

Program EVALPLOT
(Version 2021-1)

by

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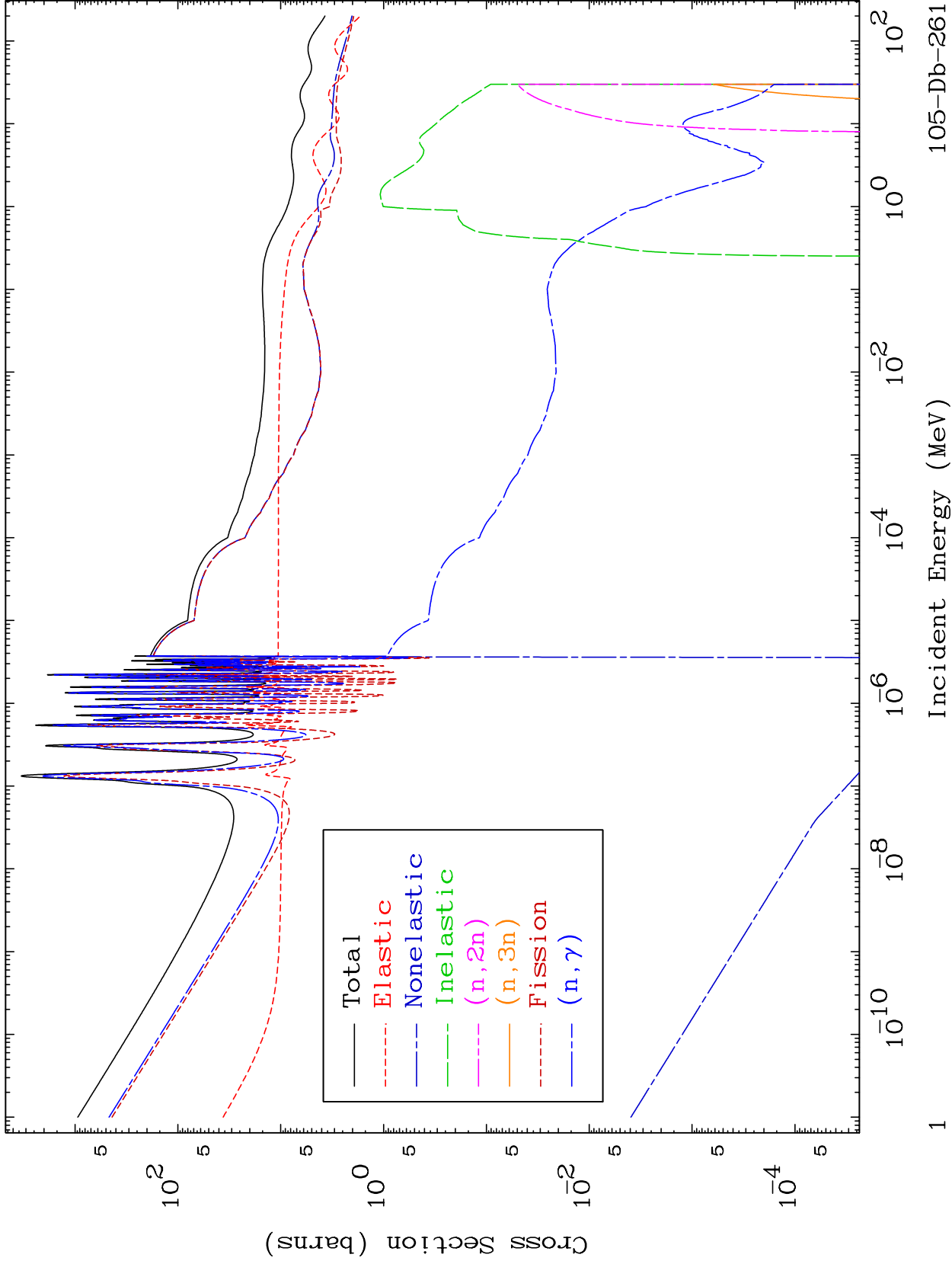
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Press Mouse Button to Start

MAT 561

Neutron Major
293 Kelvin Cross Sections

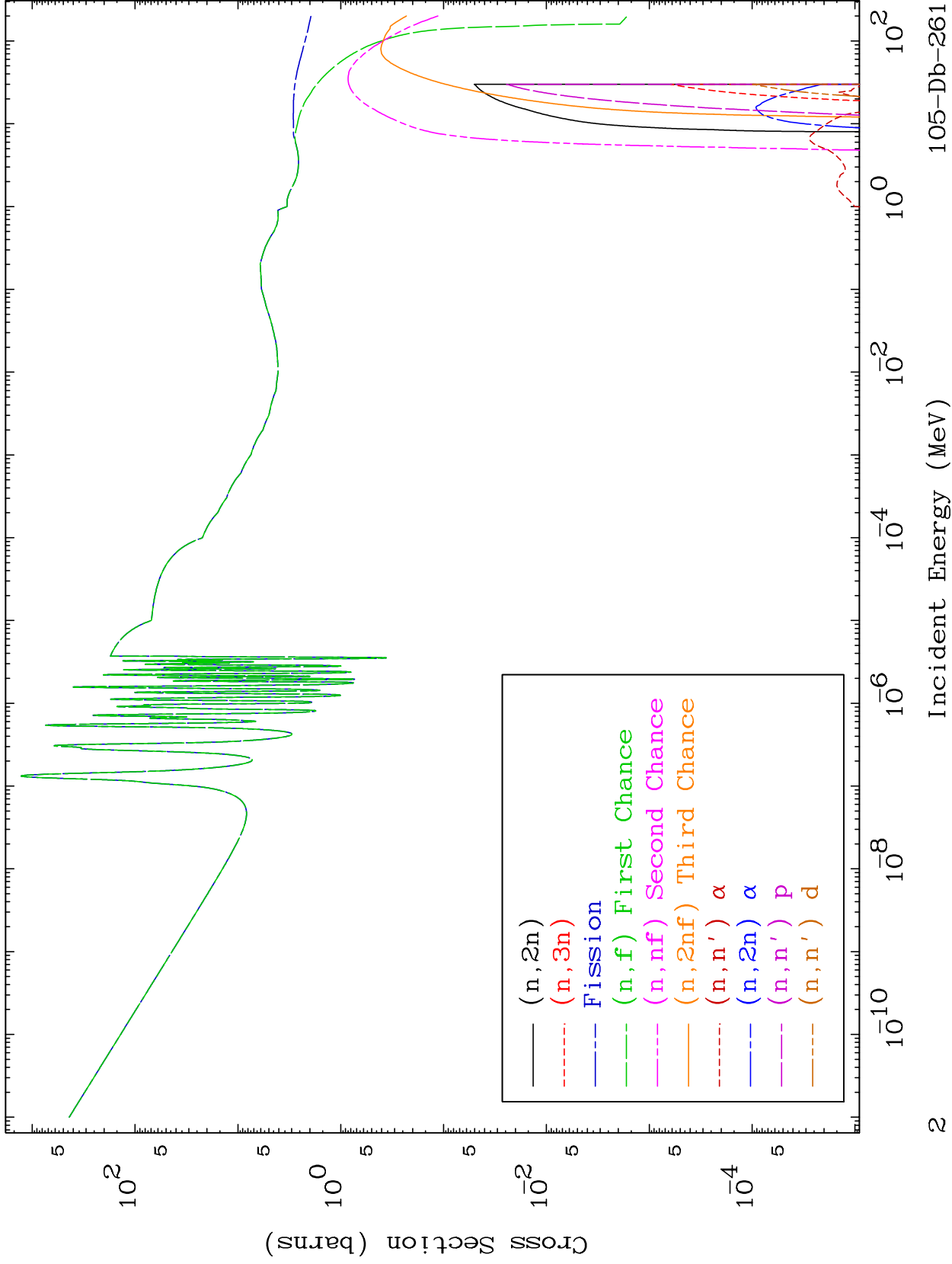
105-Db-261

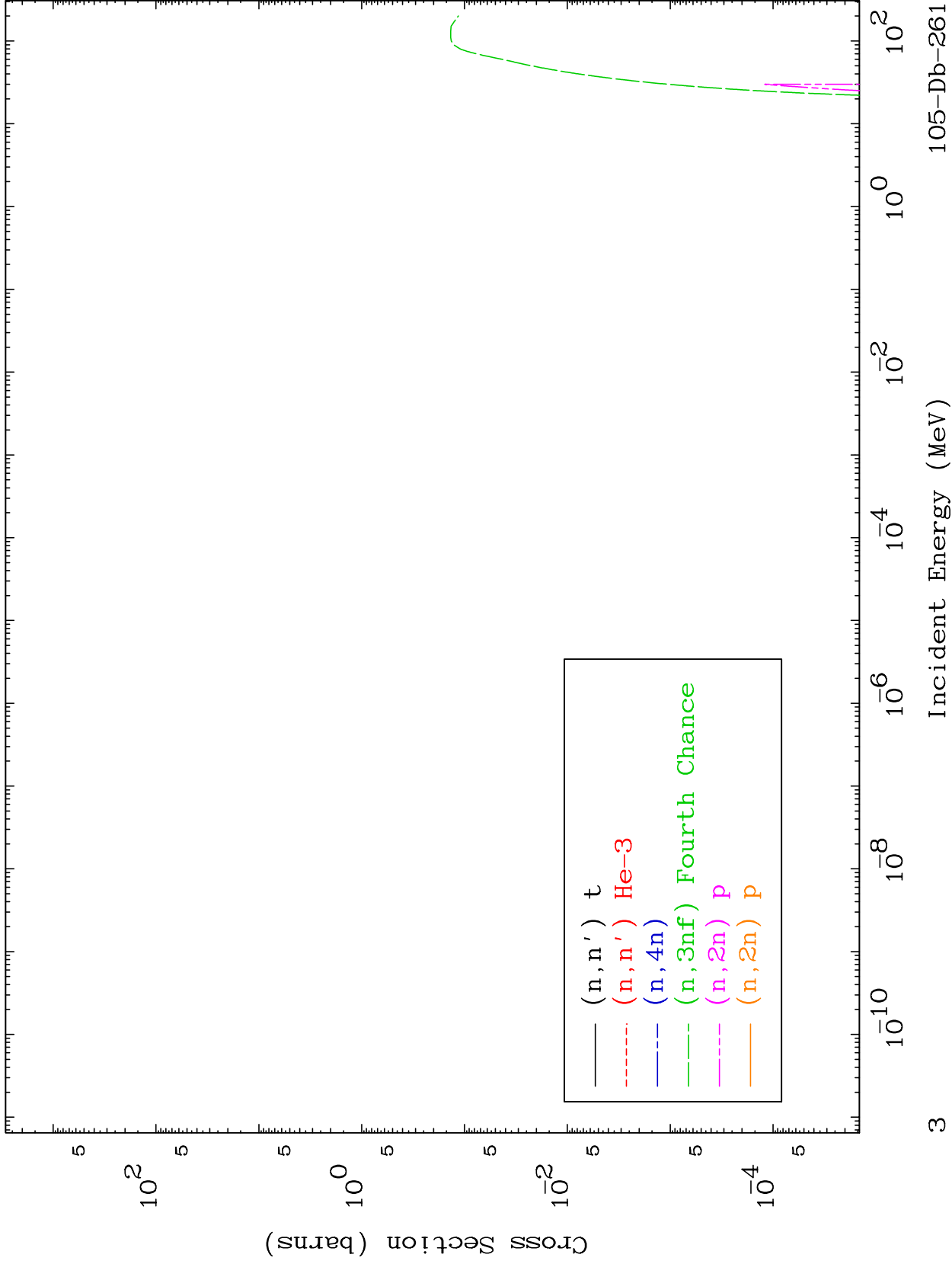


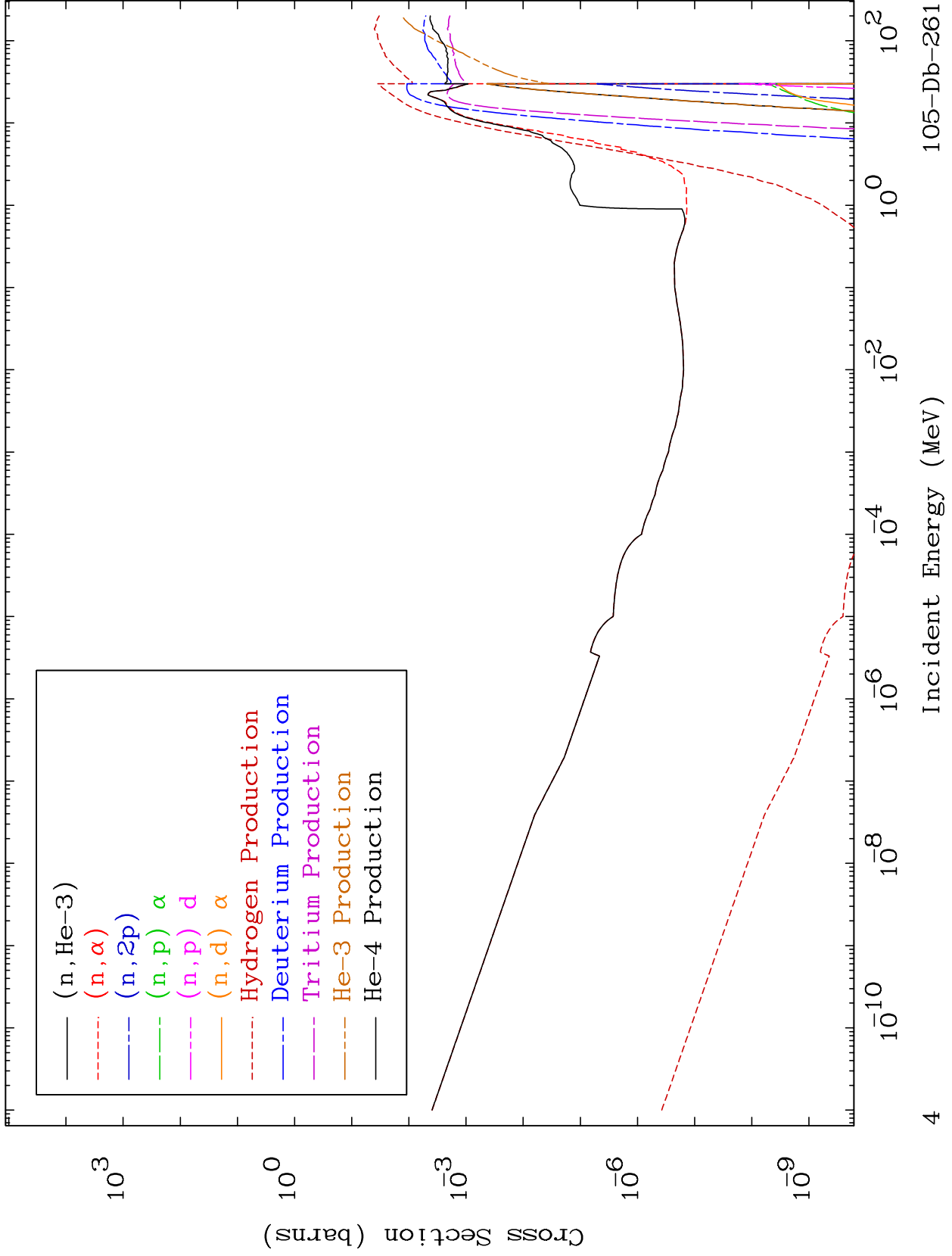
MAT 561

Neutron Absorption
293 Kelvin Cross Sections

105-Db-261



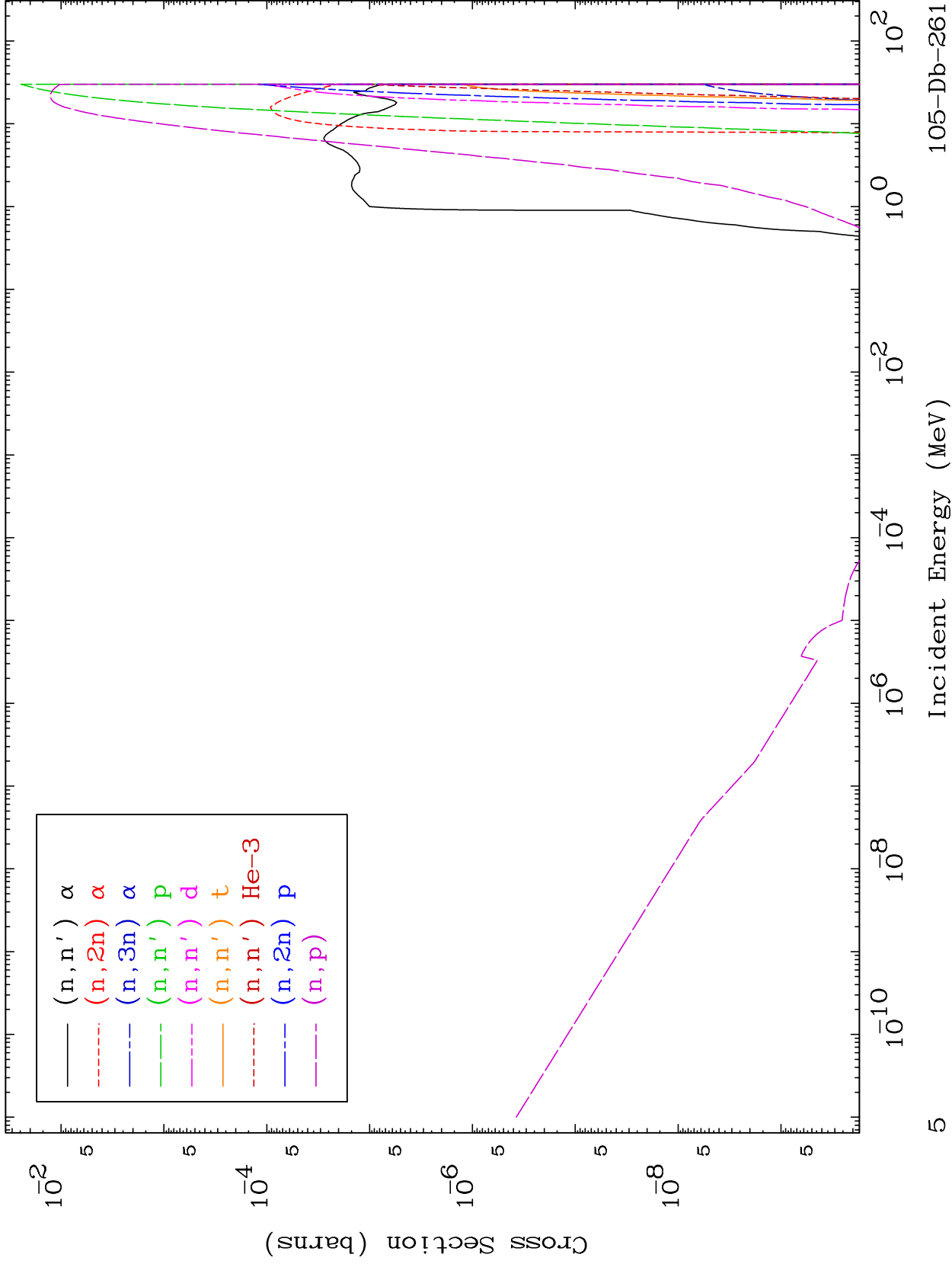




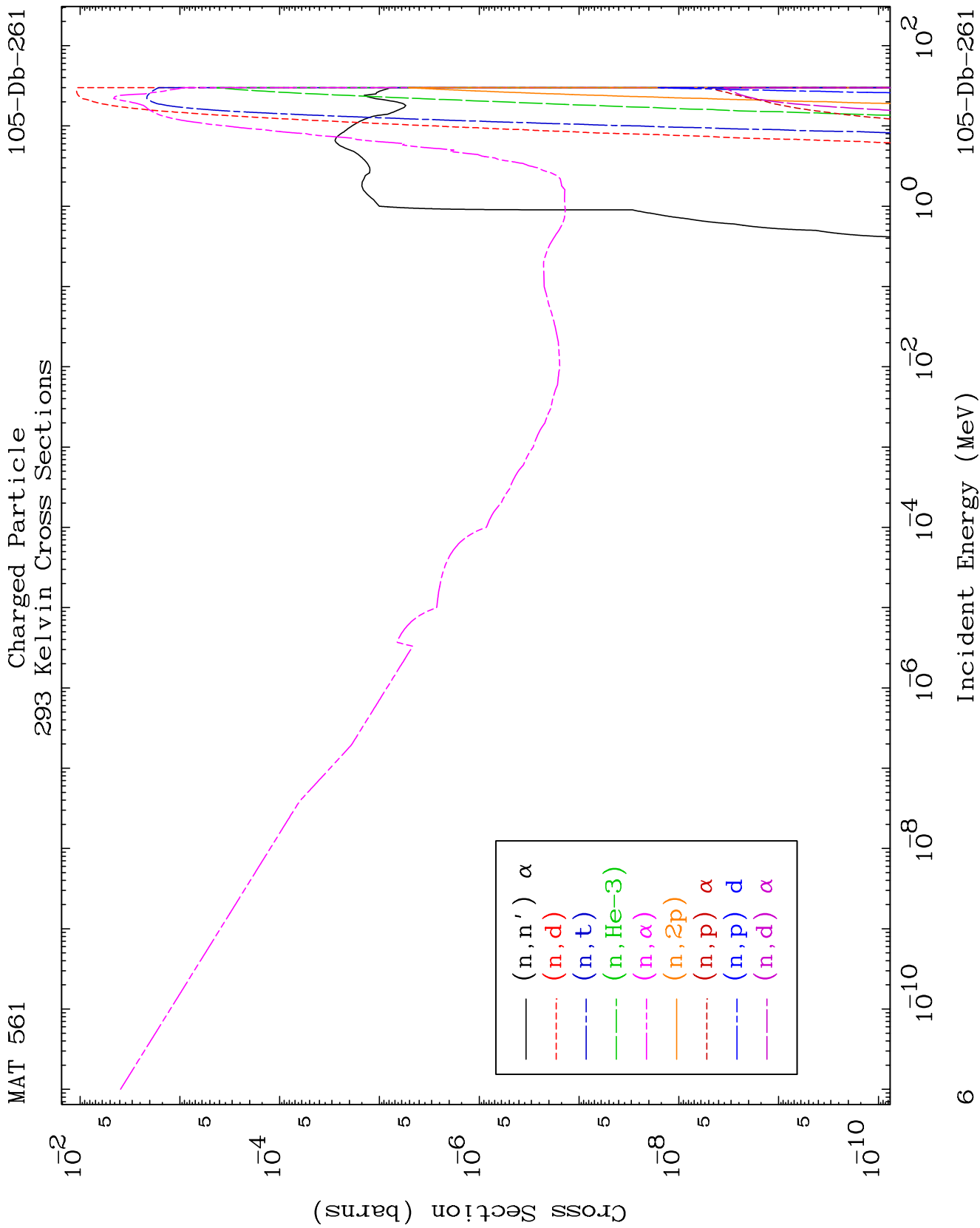
MAT 561

Charged Particle
293 Kelvin Cross Sections

105-Db-261



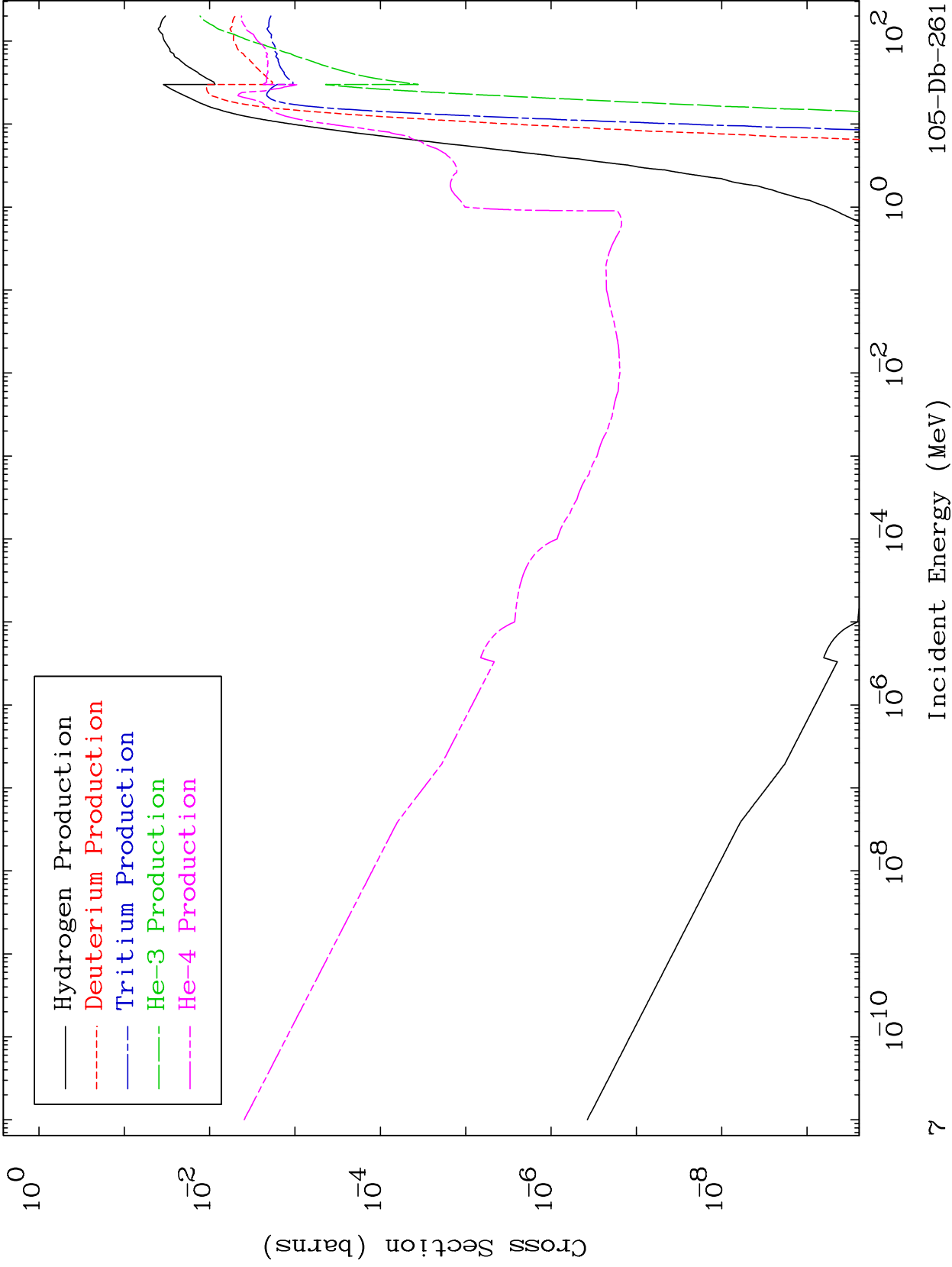
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MAT 561

Particle Production
293 Kelvin Cross Sections

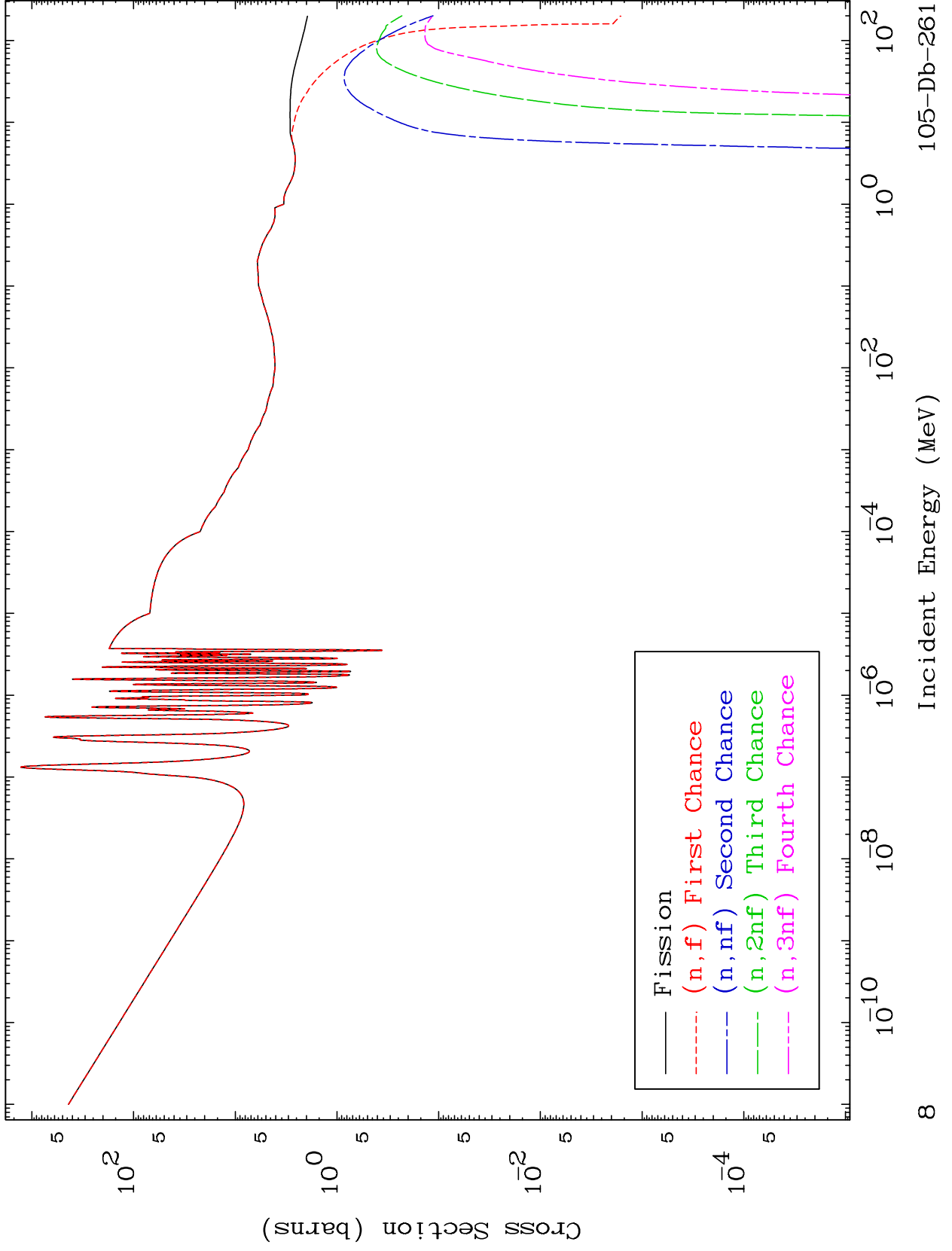
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MAT 561

Fission
293 Kelvin Cross Sections

105-Db-261

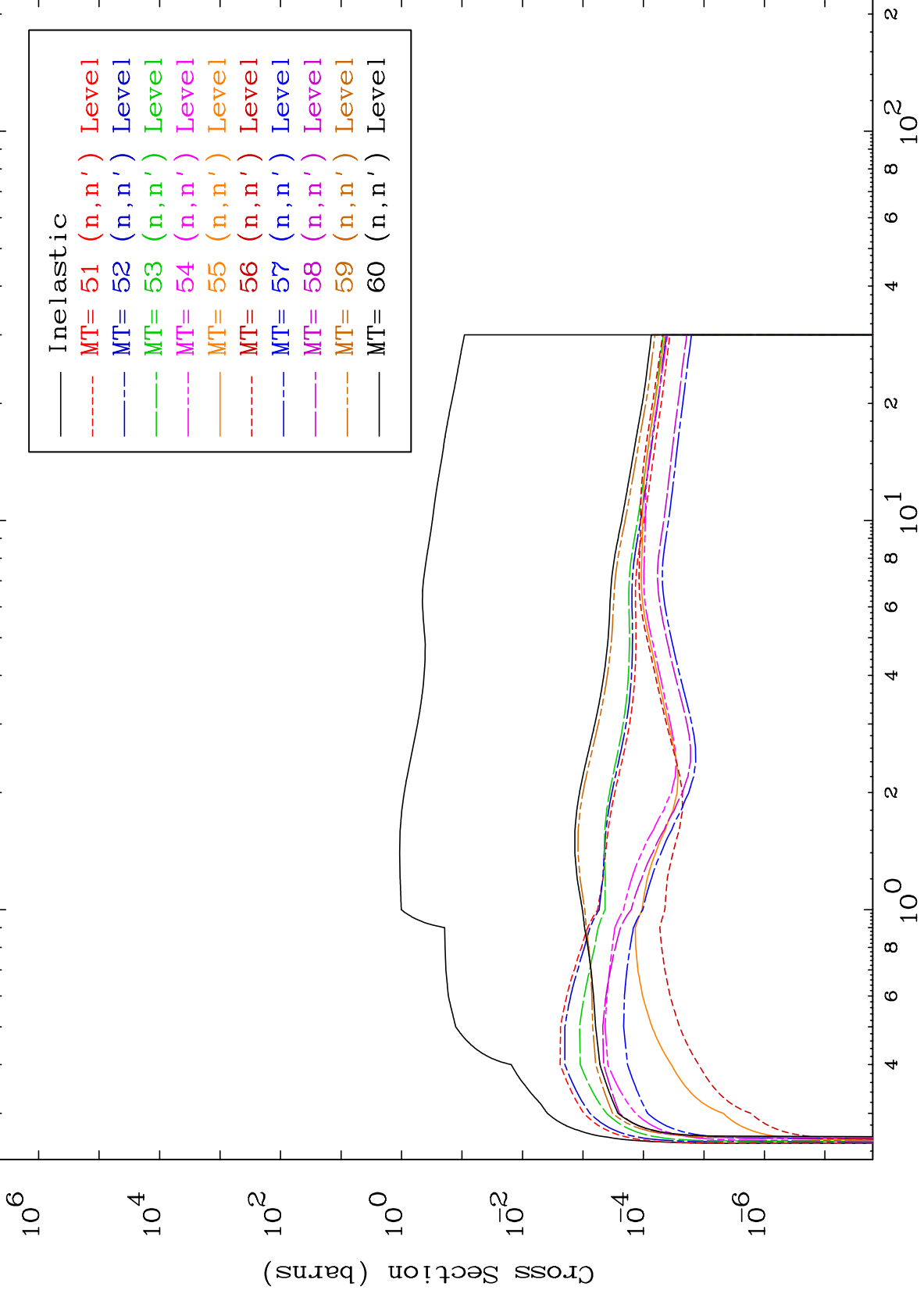


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(n,n') Levels

105-Db-261

293 Kelvin Cross Sections

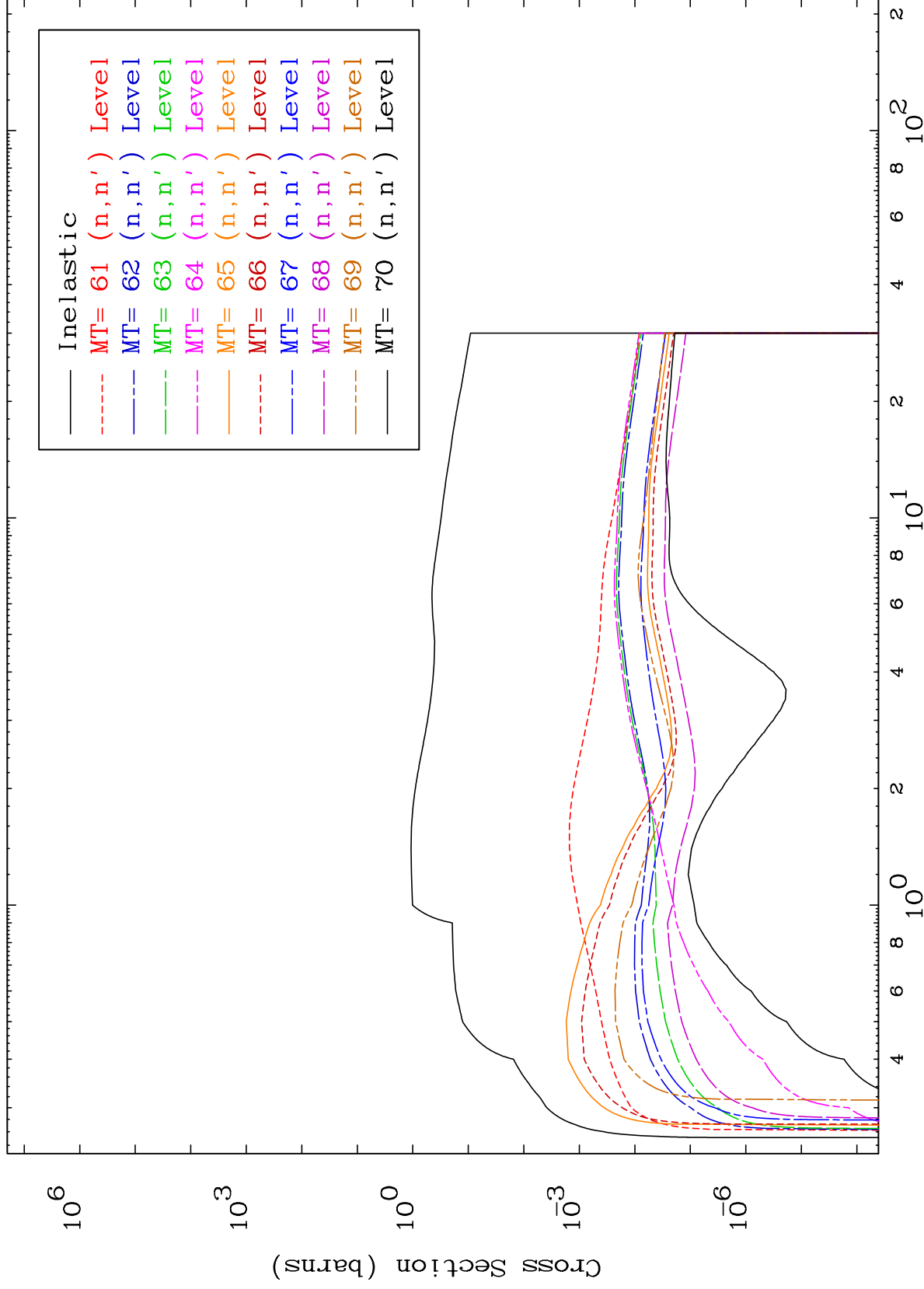


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(n,n') Levels

293 Kelvin Cross Sections

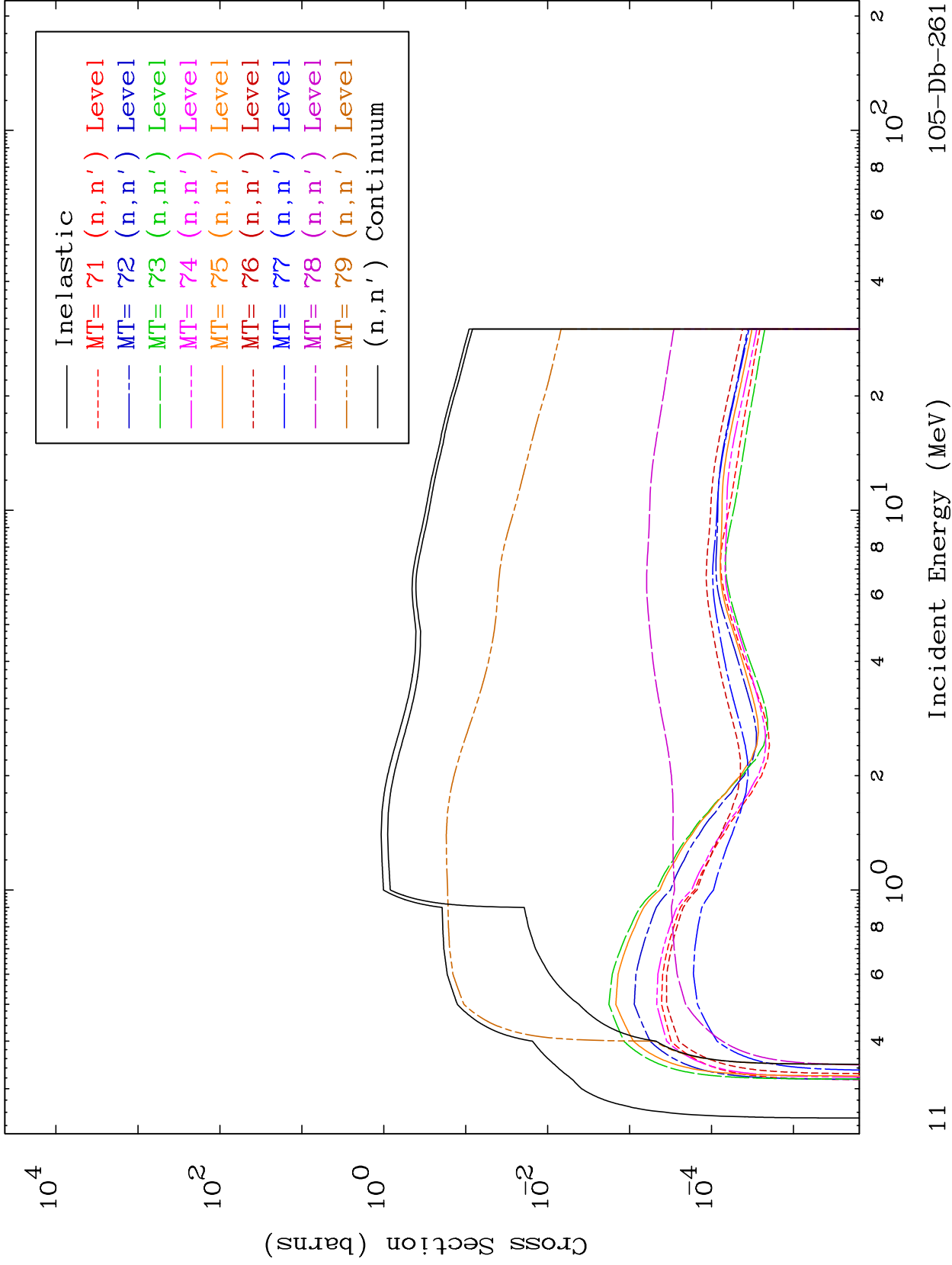
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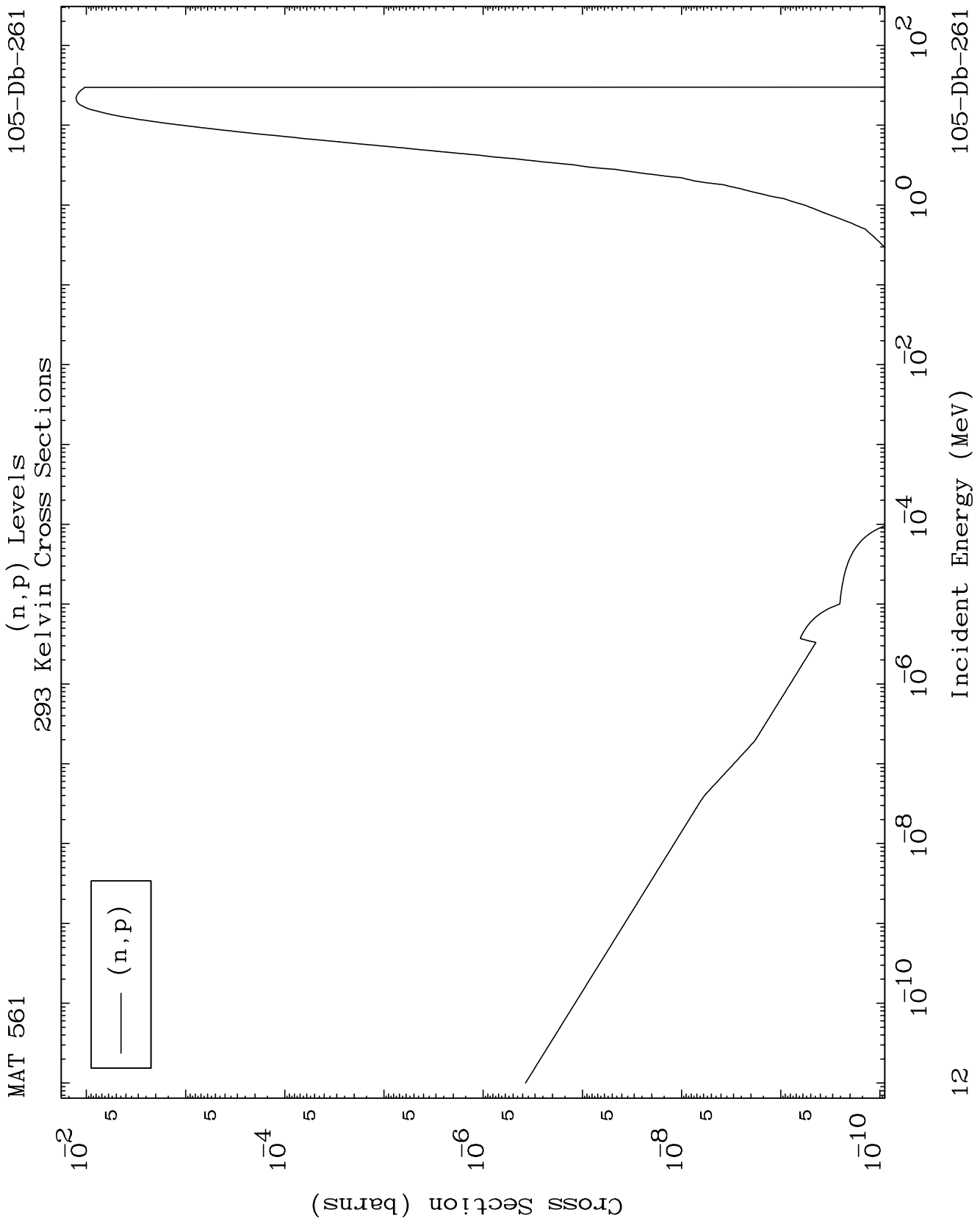


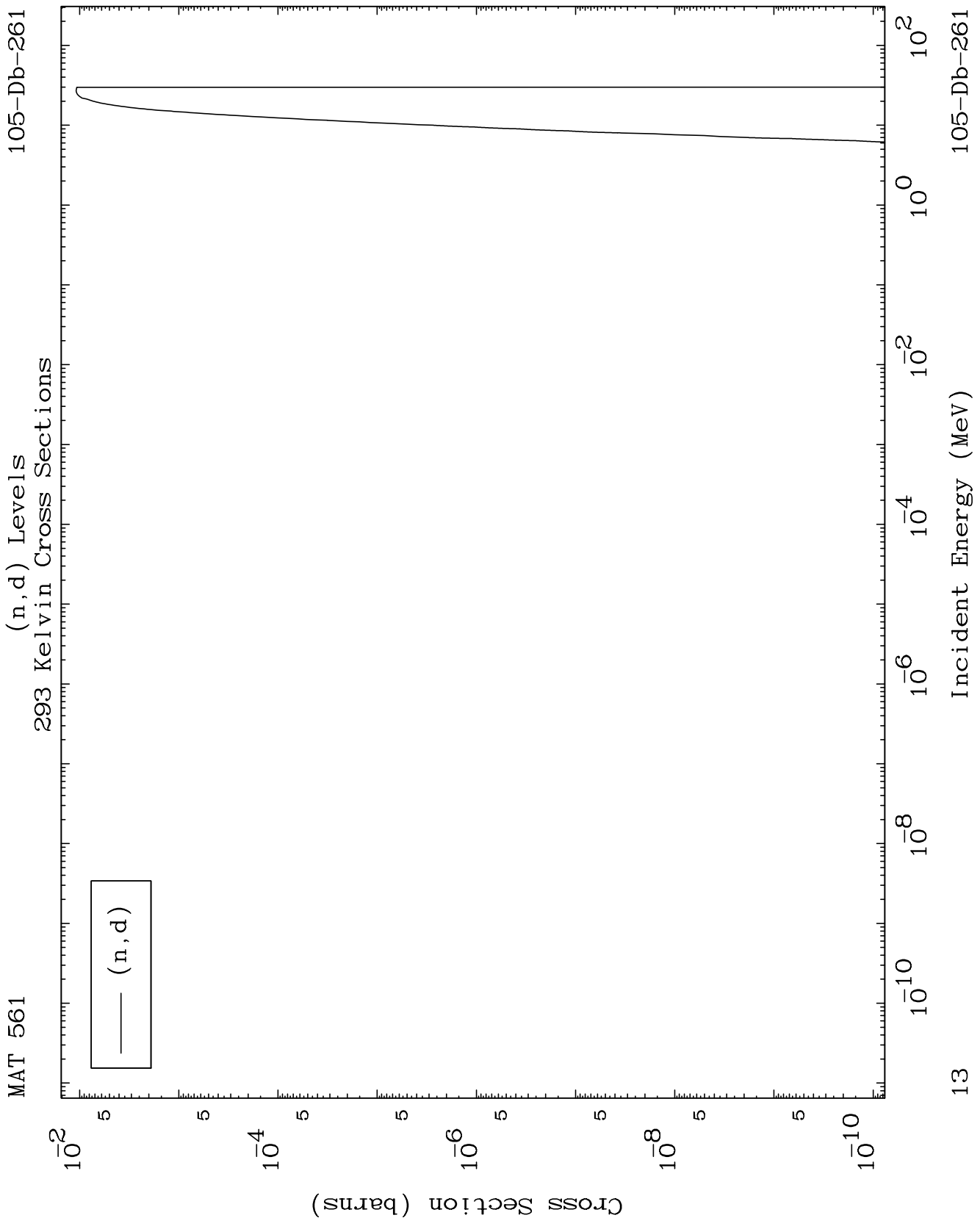
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Incident Energy (MeV)

105-Db-261



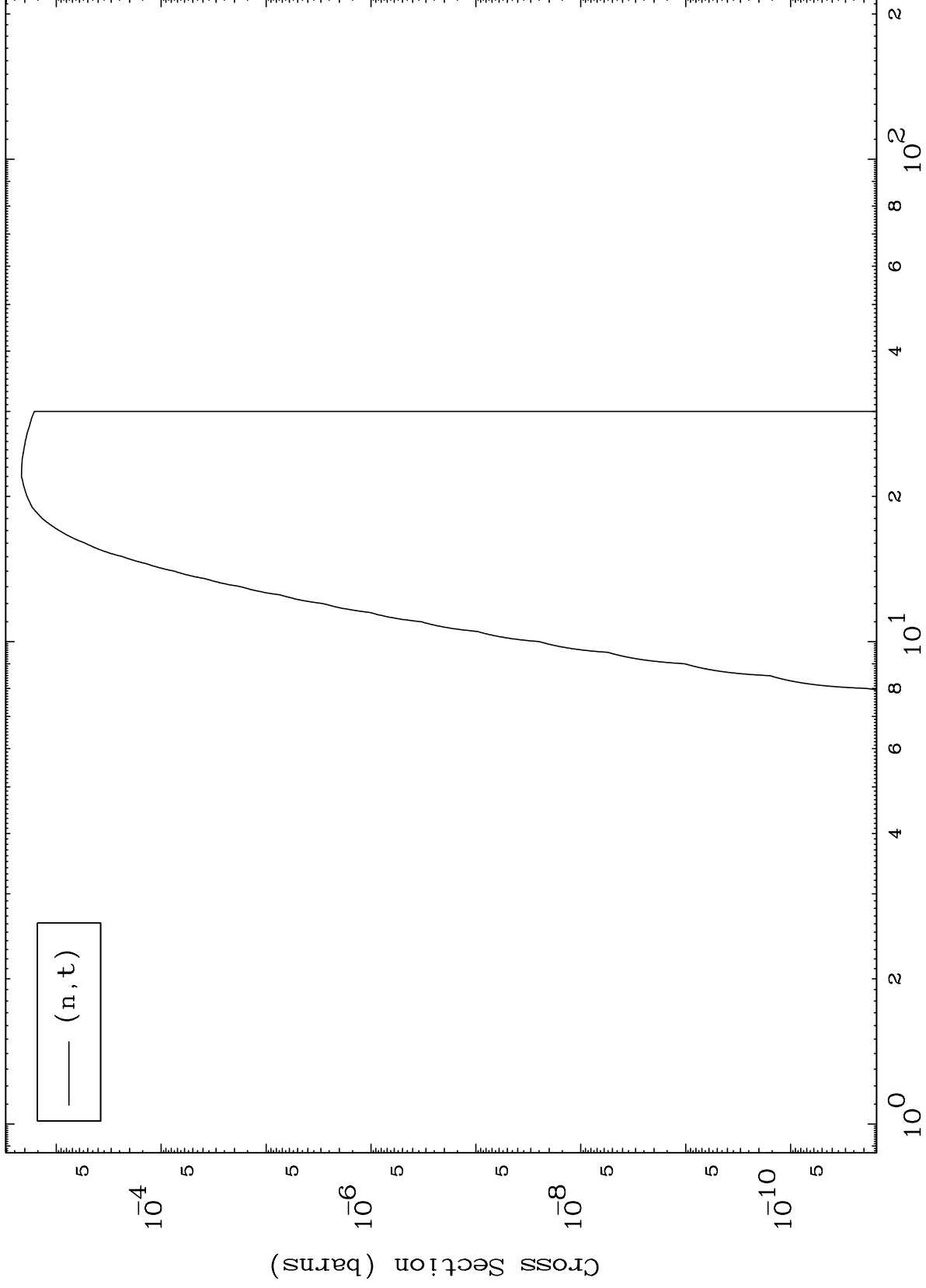




MAT 561

(n,t) Levels
293 Kelvin Cross Sections

105-Db-261



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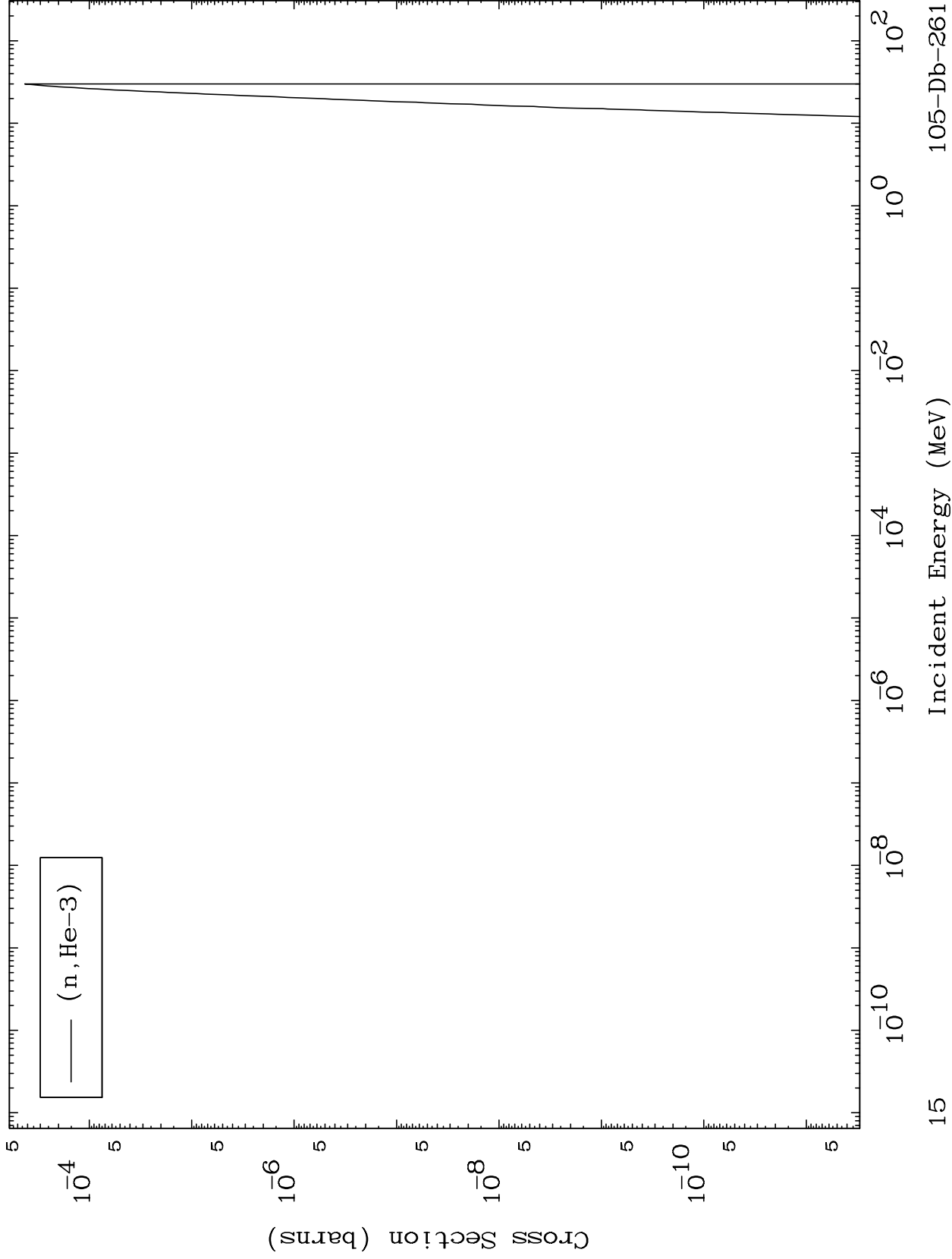
Incident Energy (MeV)

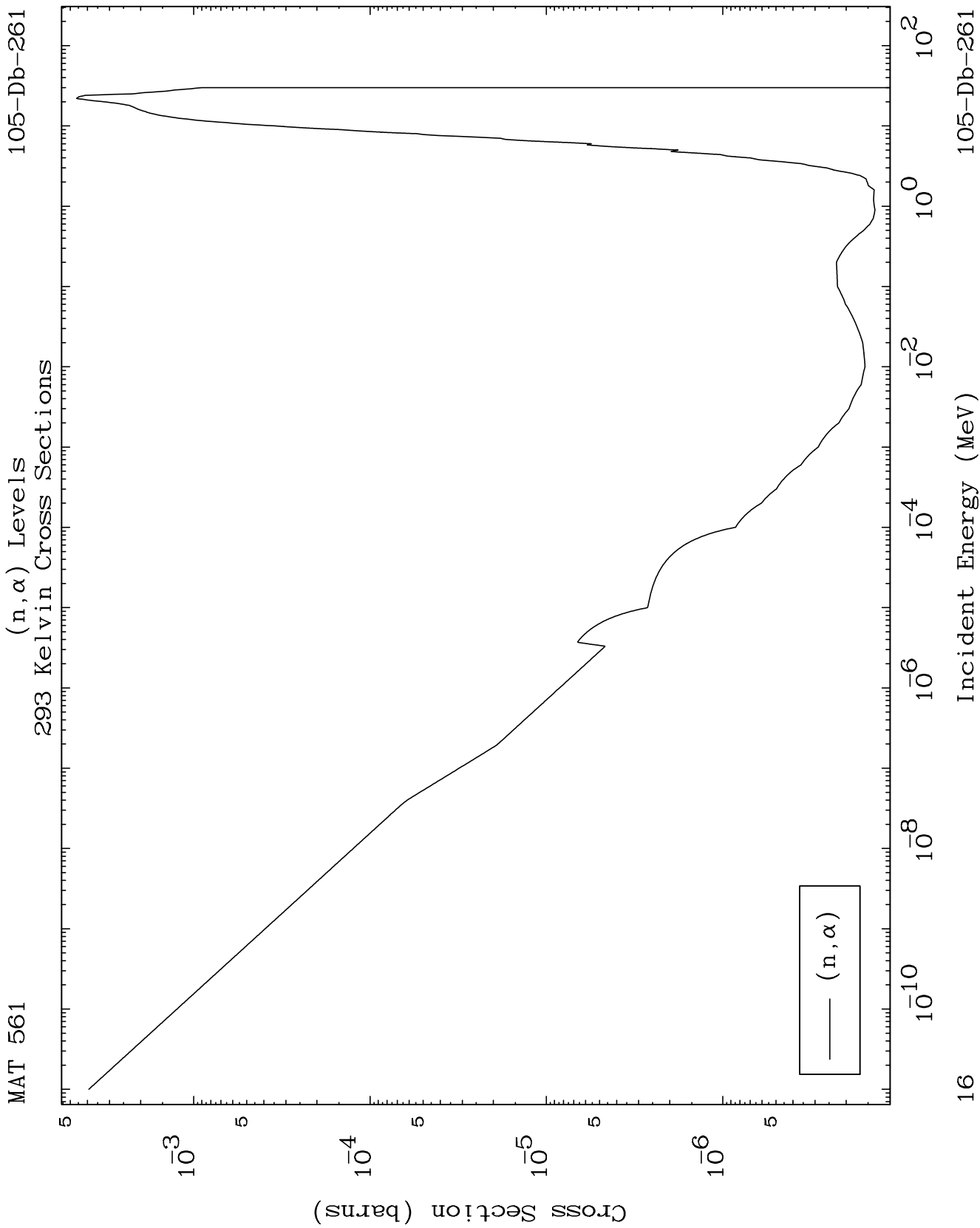
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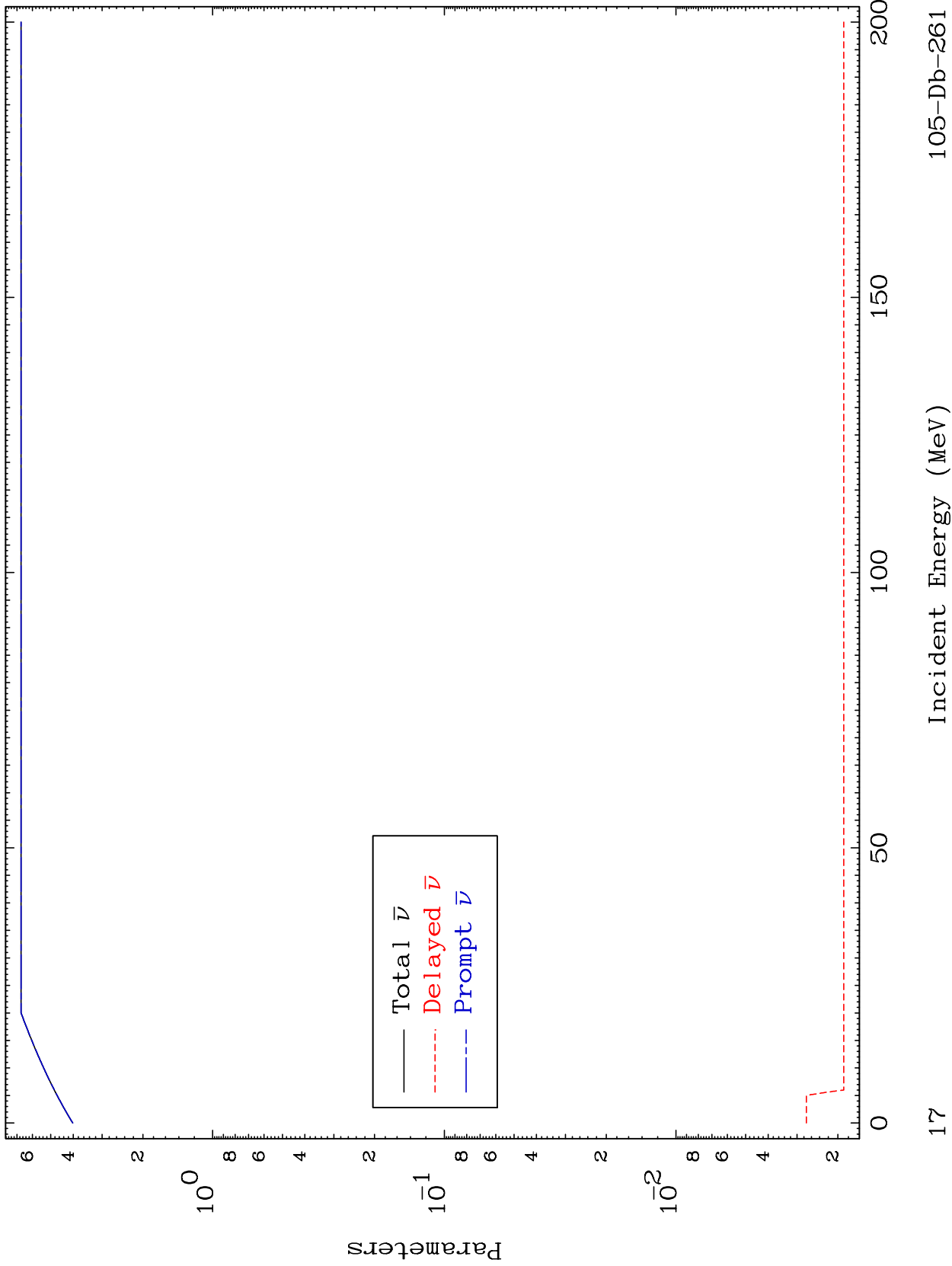
MAT 561

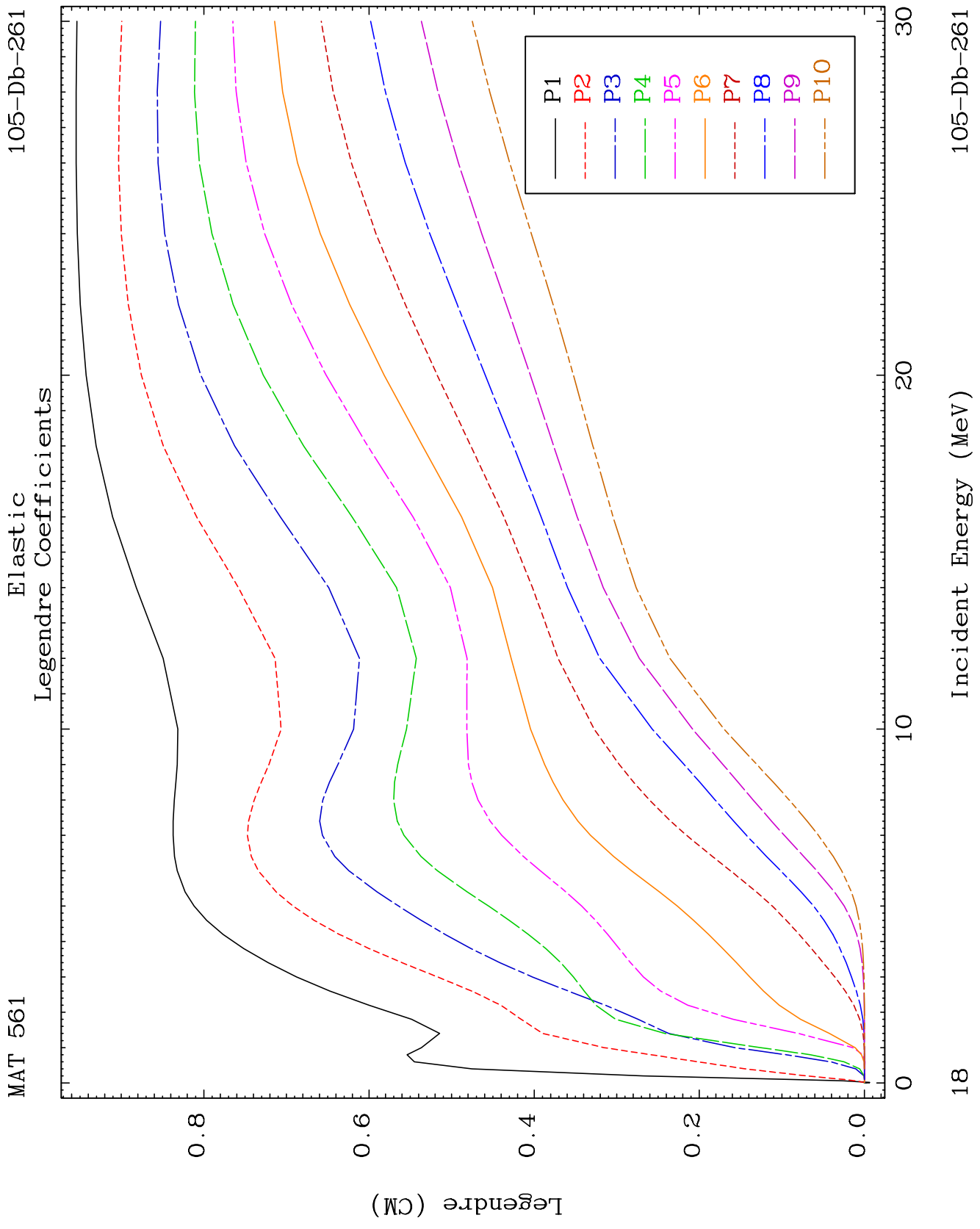
(n,He3) Levels
293 Kelvin Cross Sections

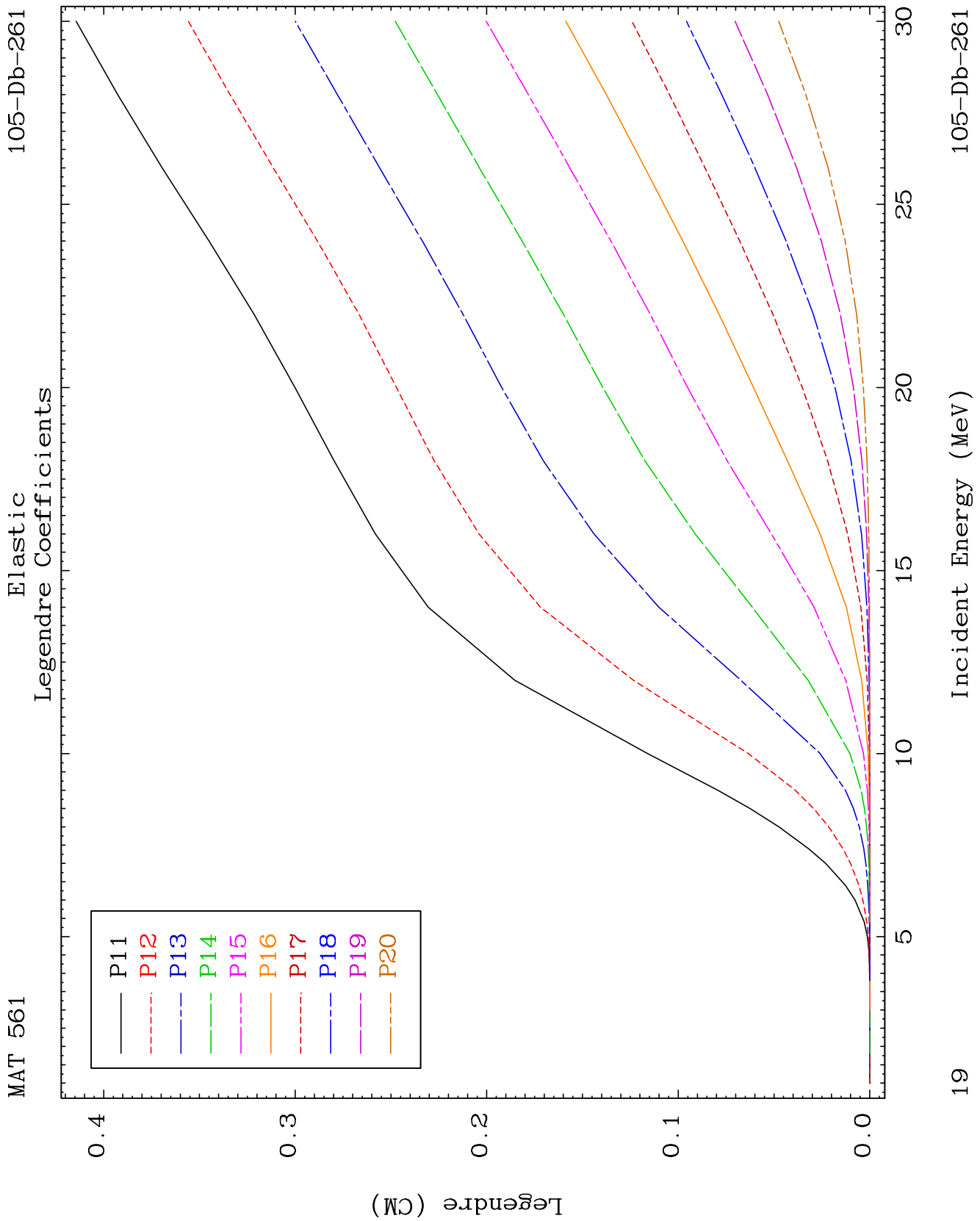
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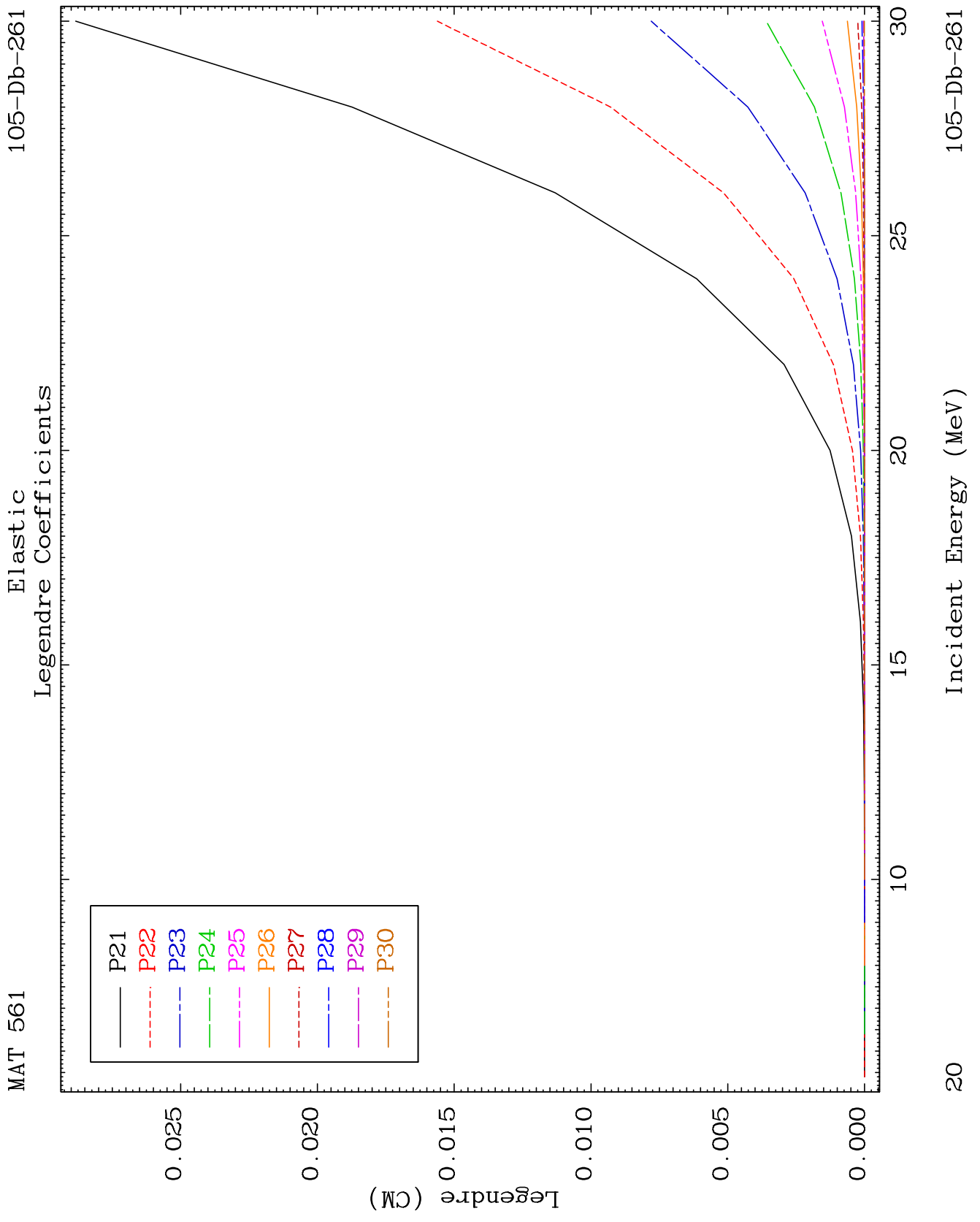








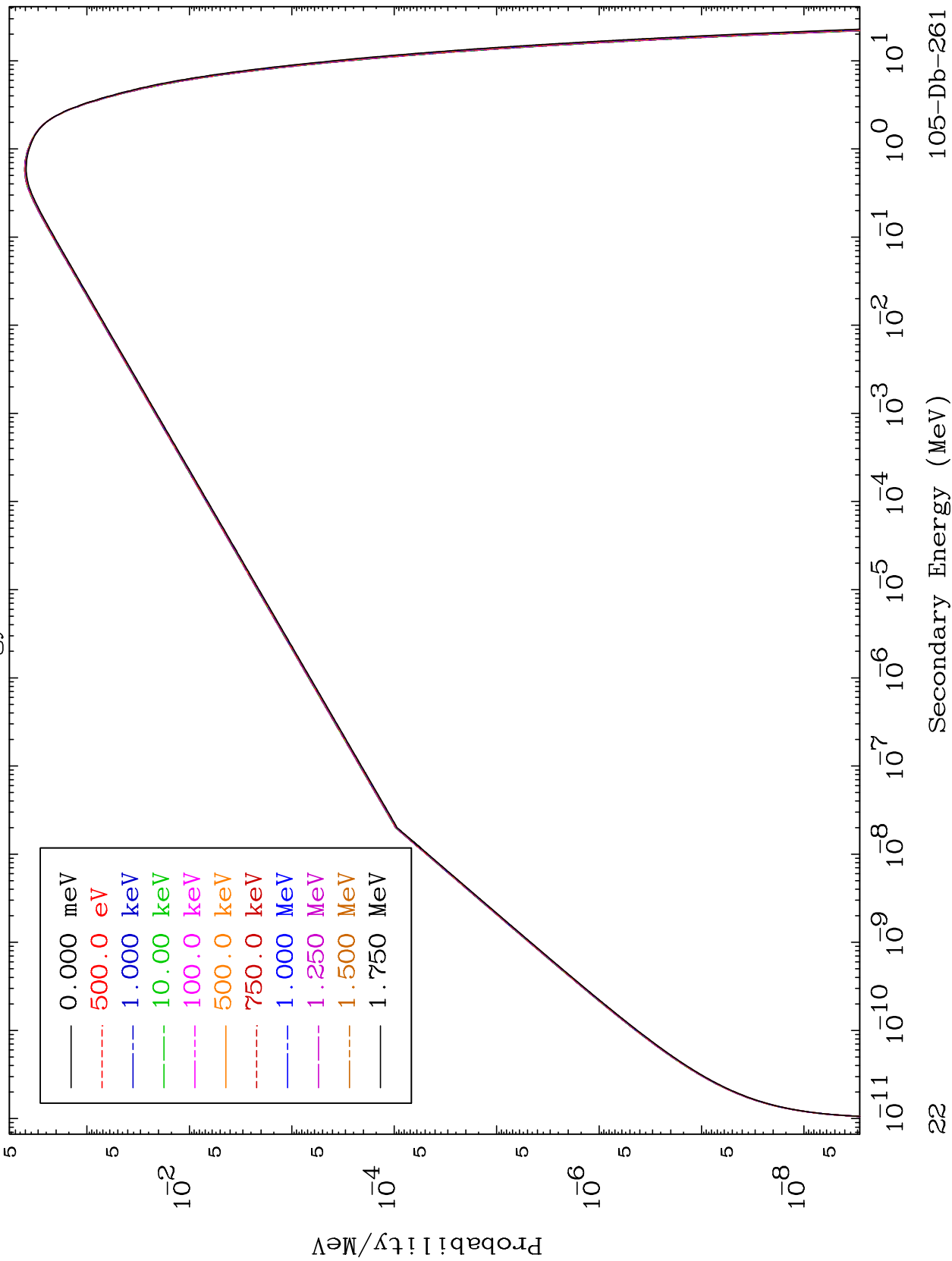




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Fission Energy Distributions

105-Db-261



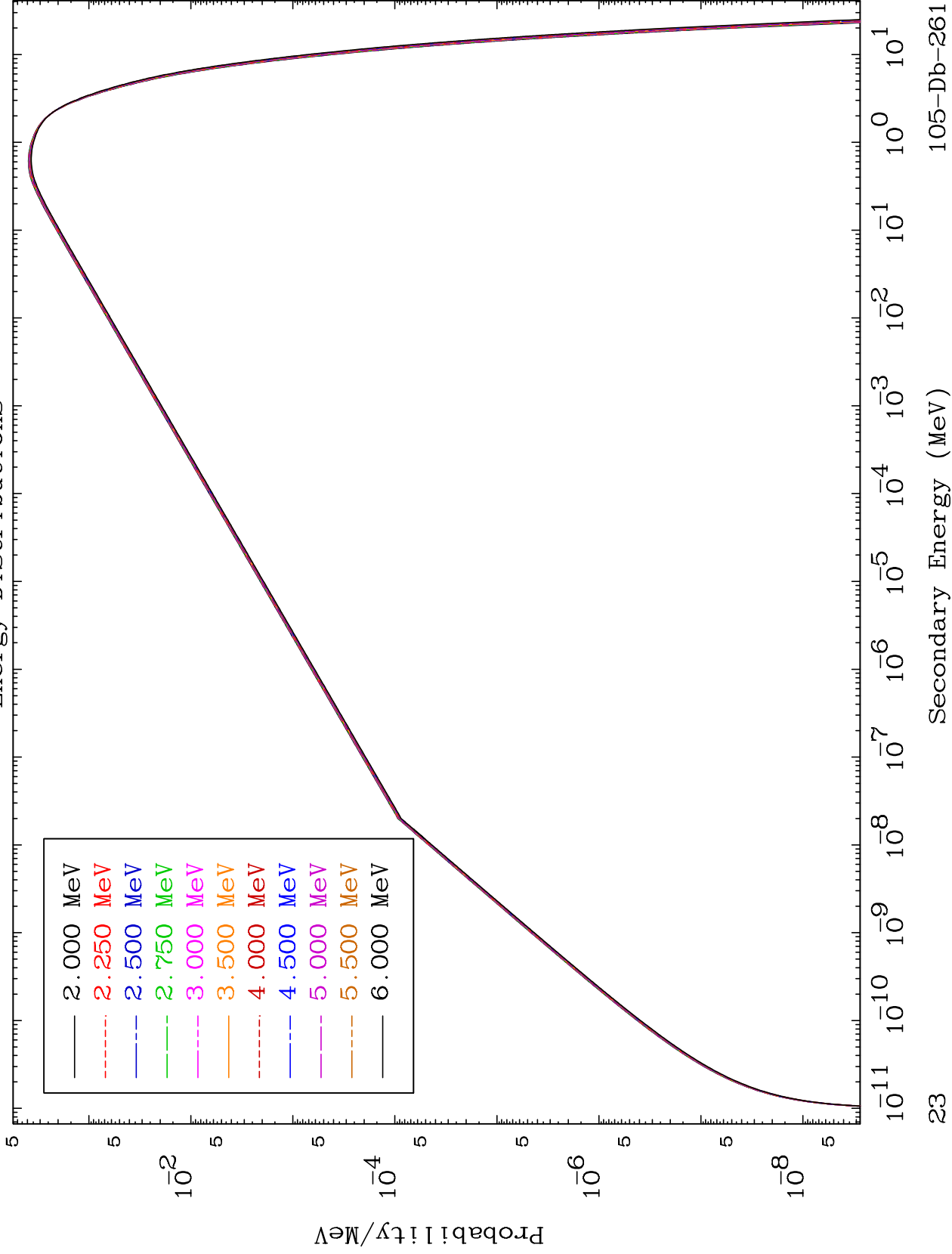
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Secondary Energy (MeV)

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Fission Energy Distributions

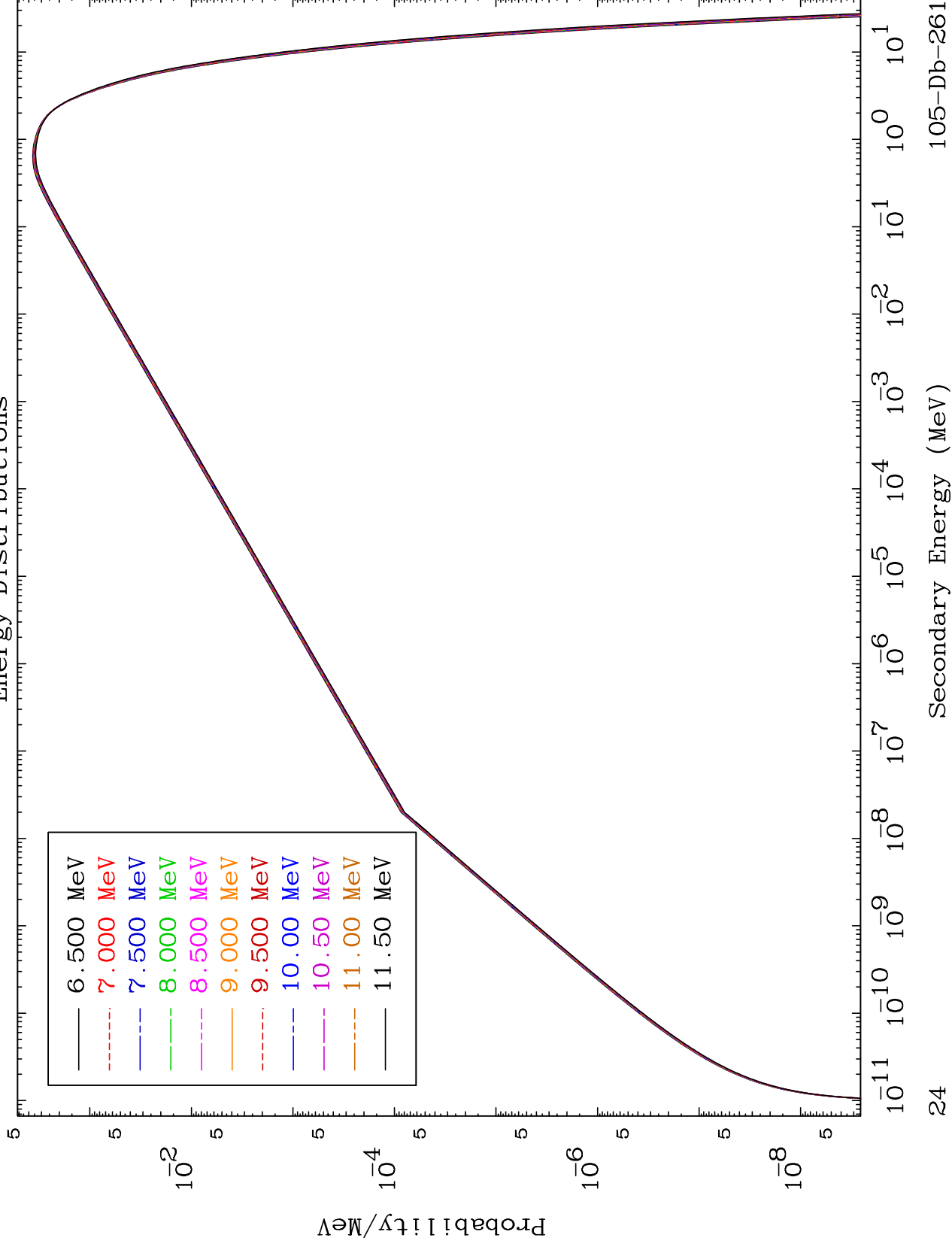
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Fission Energy Distributions

105-Db-261



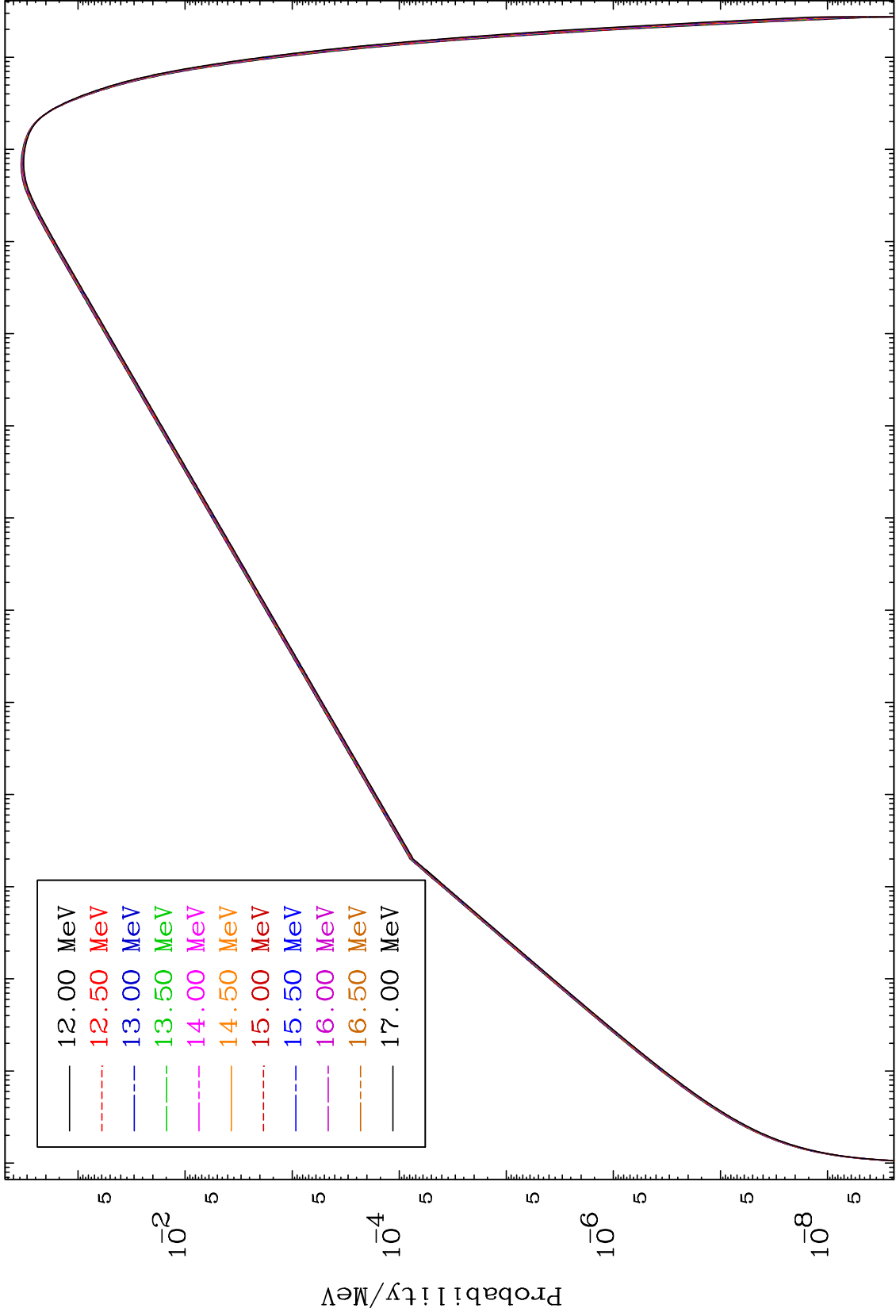
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Fission Energy Distributions

105-Db-261



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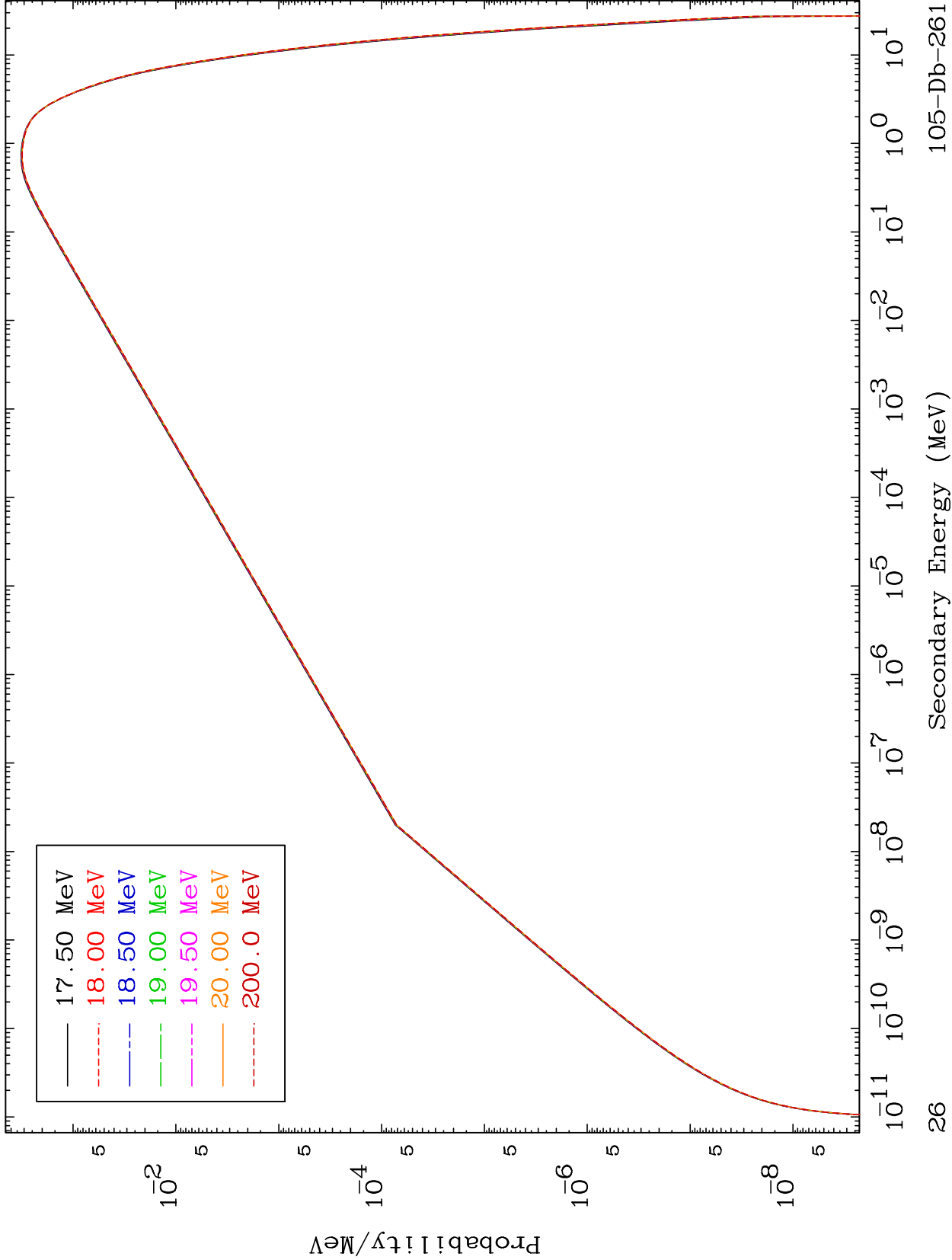
Secondary Energy (MeV)

105-Db-261

MAT 561

Fission Energy Distributions

105-Db-261



105-Db-261

Secondary Energy (MeV)

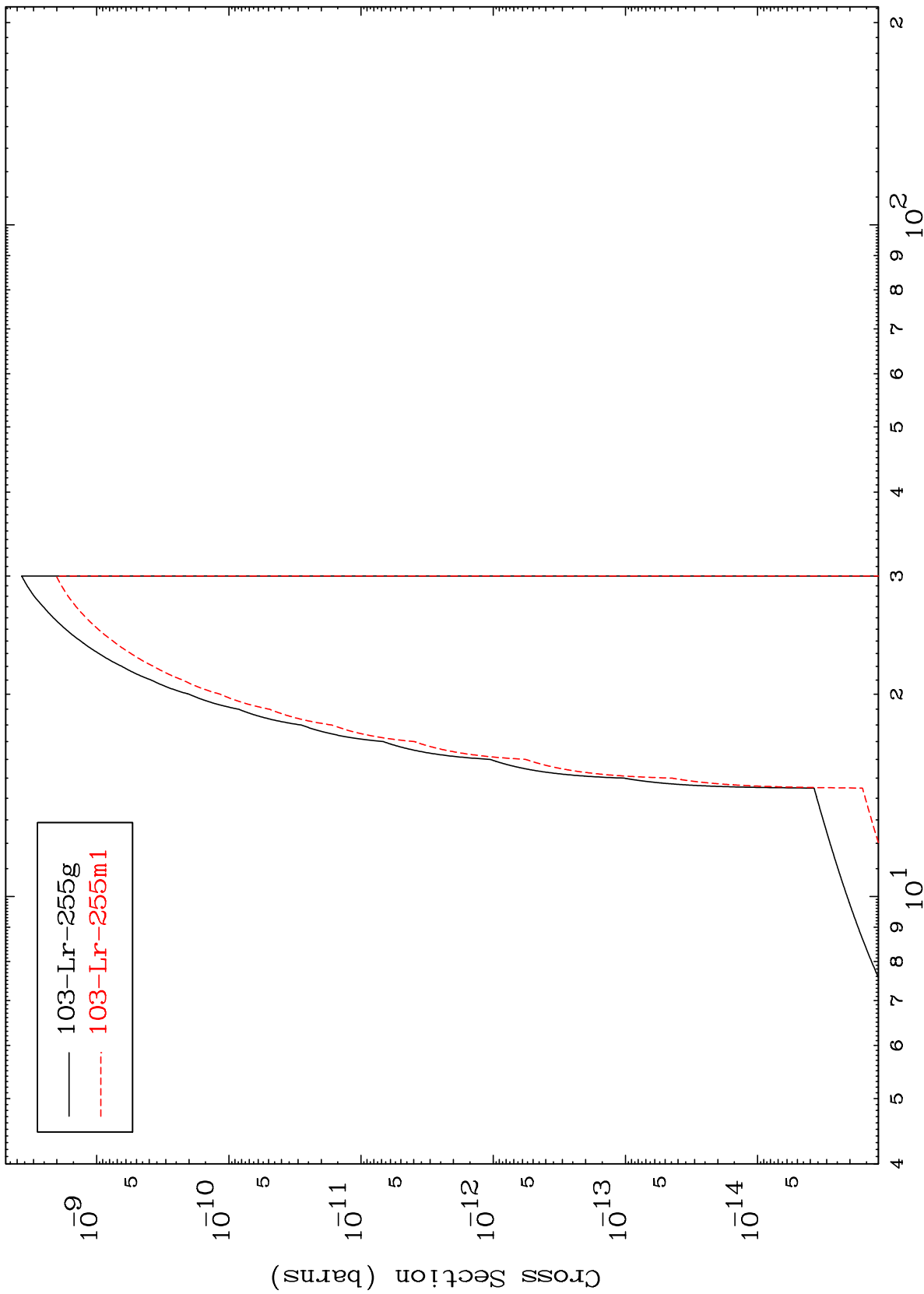
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MAT 561

(n,3n) α

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Radionuclide Production Cross Section



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Incident Energy (MeV)

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