

Program EVALPLOT
(Version 2021-1)

by

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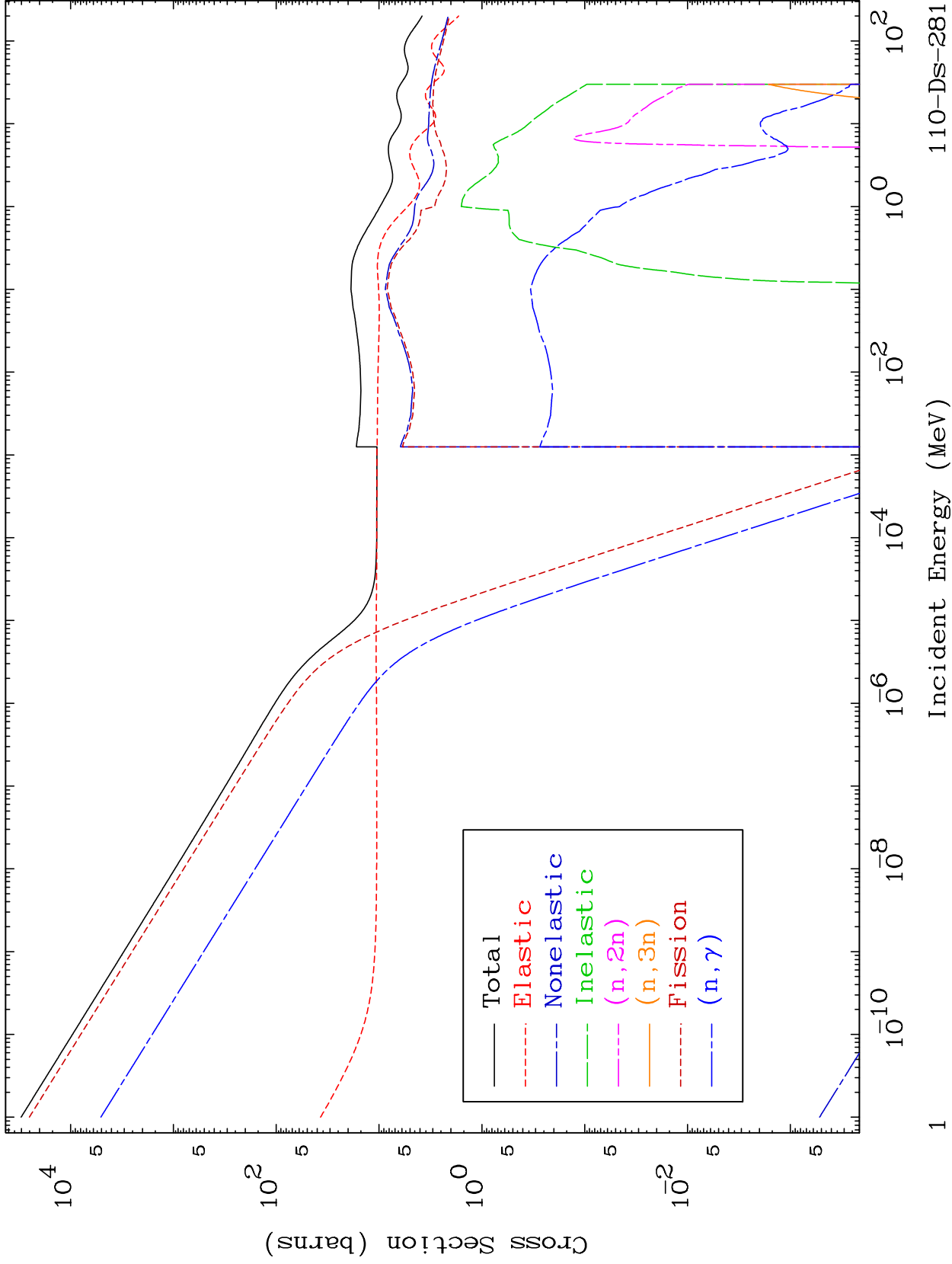
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Press Mouse Button to Start

MAT 1081

Neutron Major
293 Kelvin Cross Sections

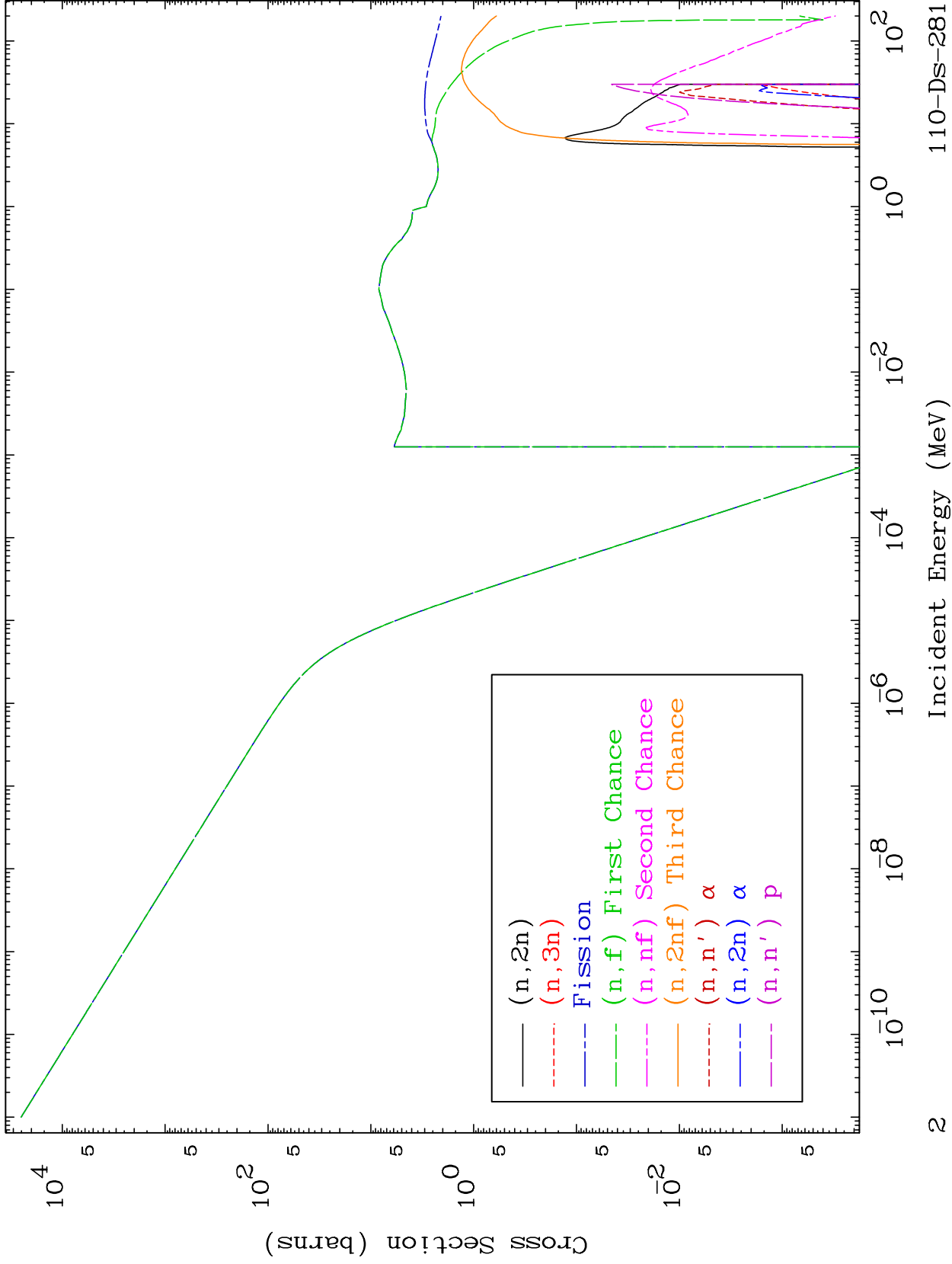
110-Ds-281



MAT 1081

Neutron Absorption
293 Kelvin Cross Sections

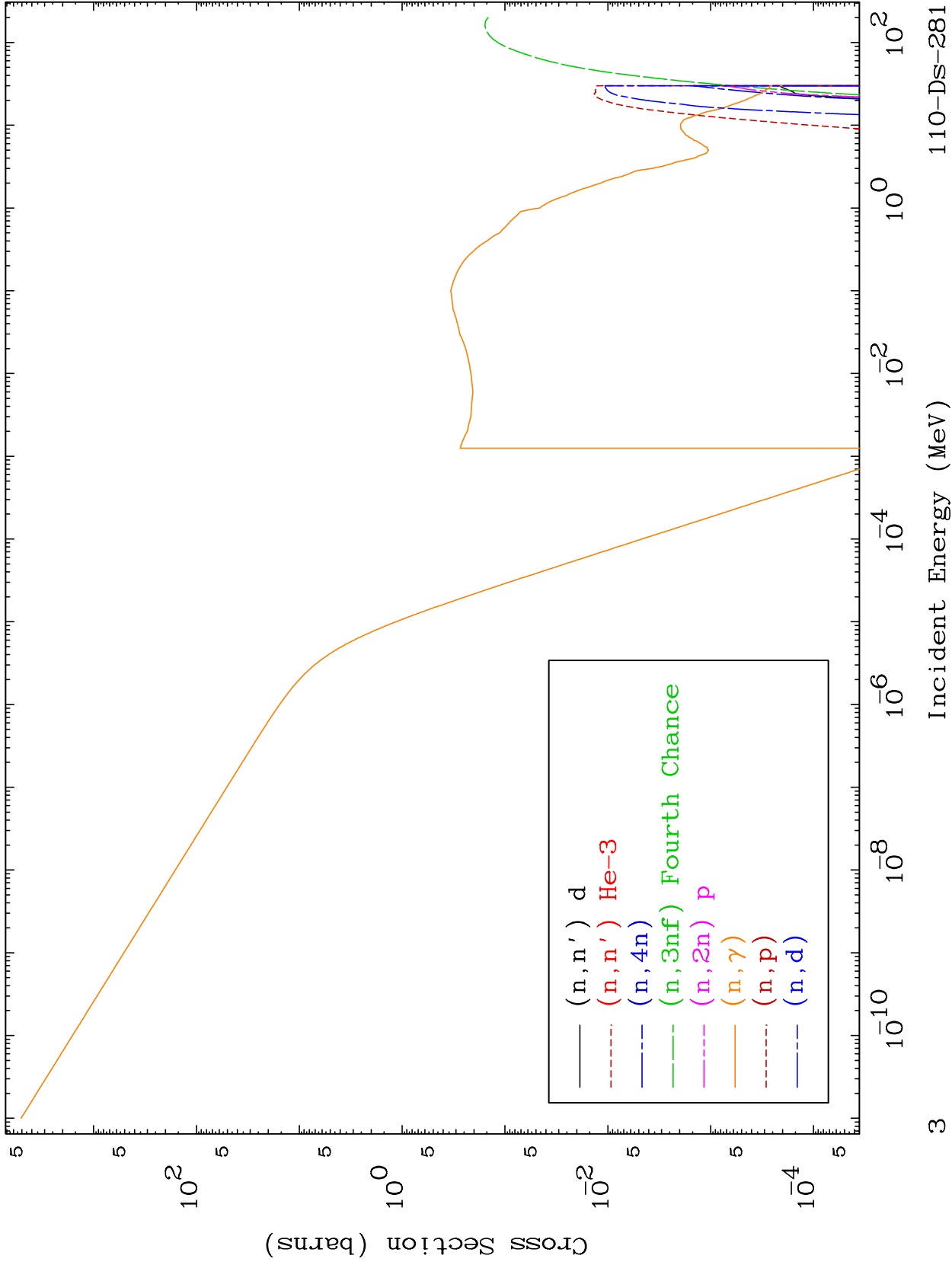
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MAT 1081

Neutron Absorption
293 Kelvin Cross Sections

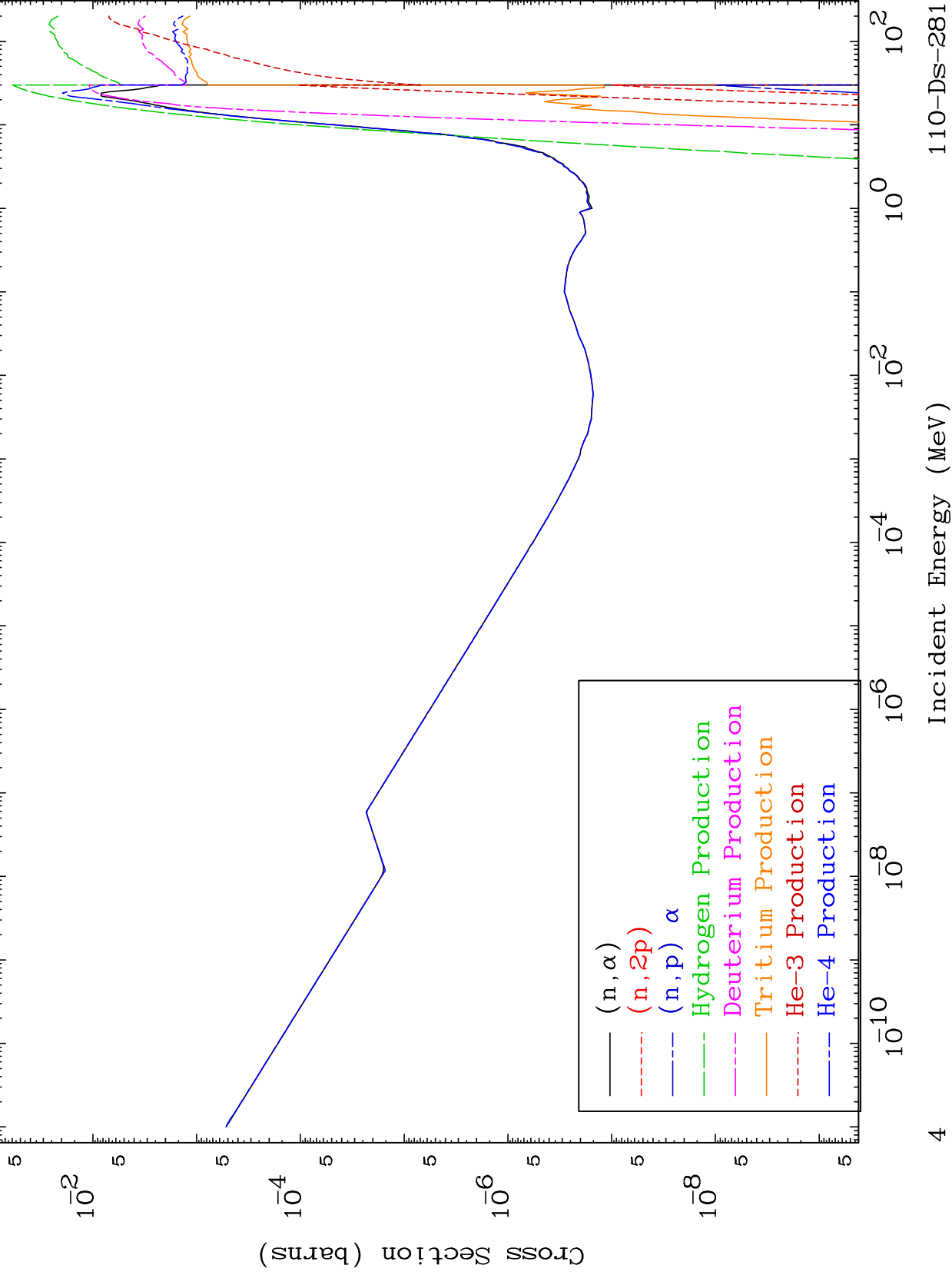
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Neutron Absorption
293 Kelvin Cross Sections

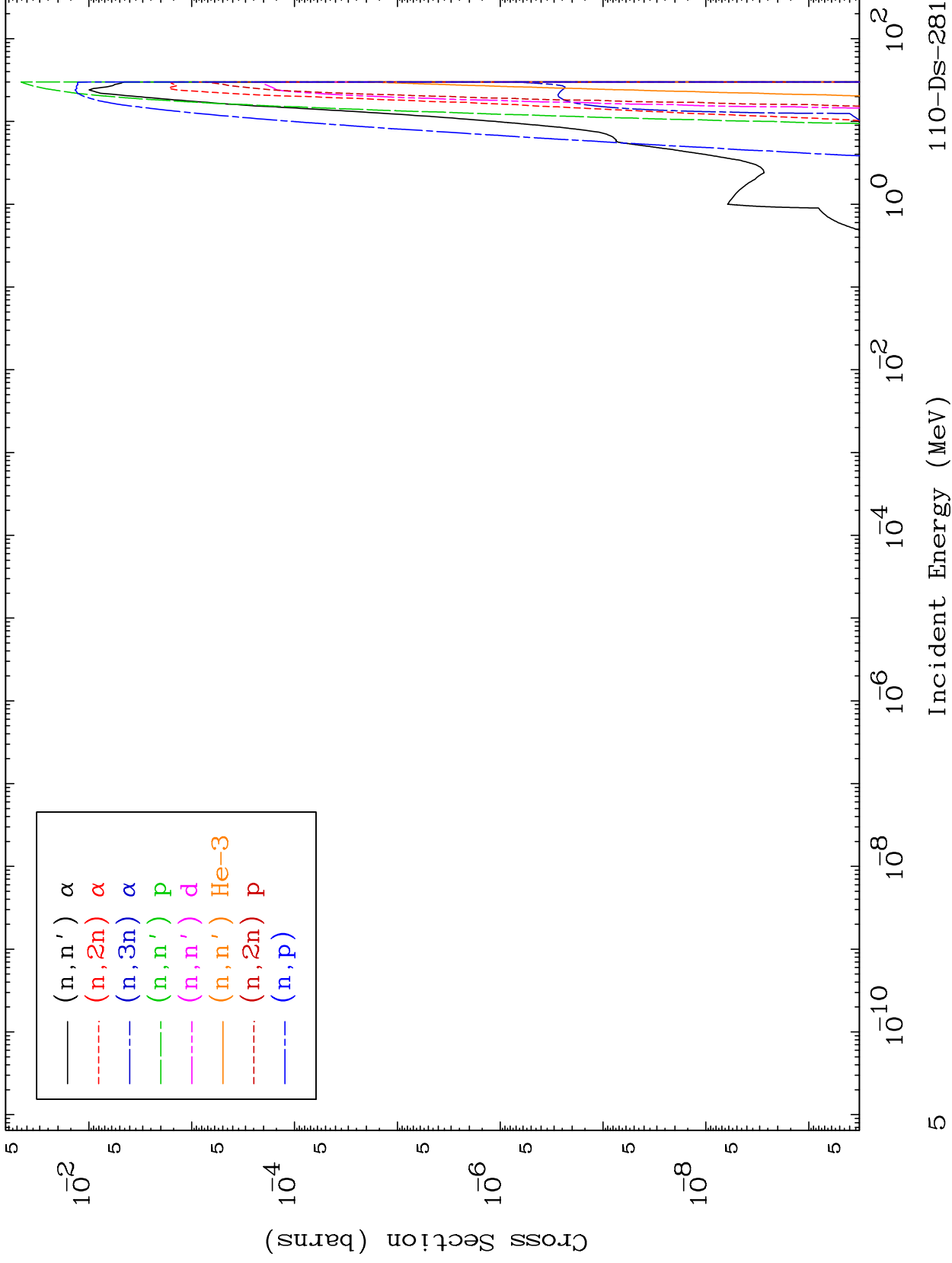
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MAT 1081

Charged Particle
293 Kelvin Cross Sections

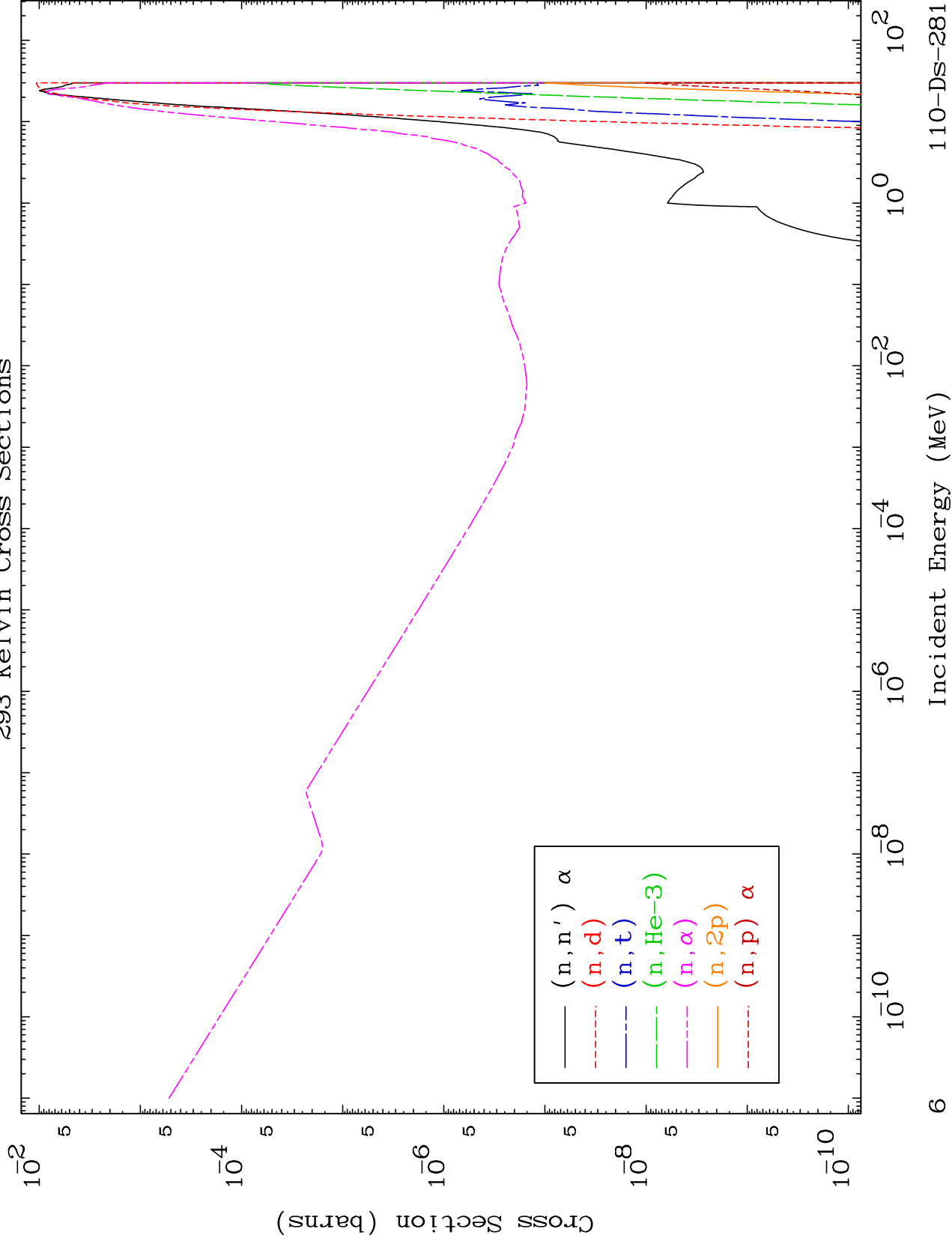
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Charged Particle
293 Kelvin Cross Sections

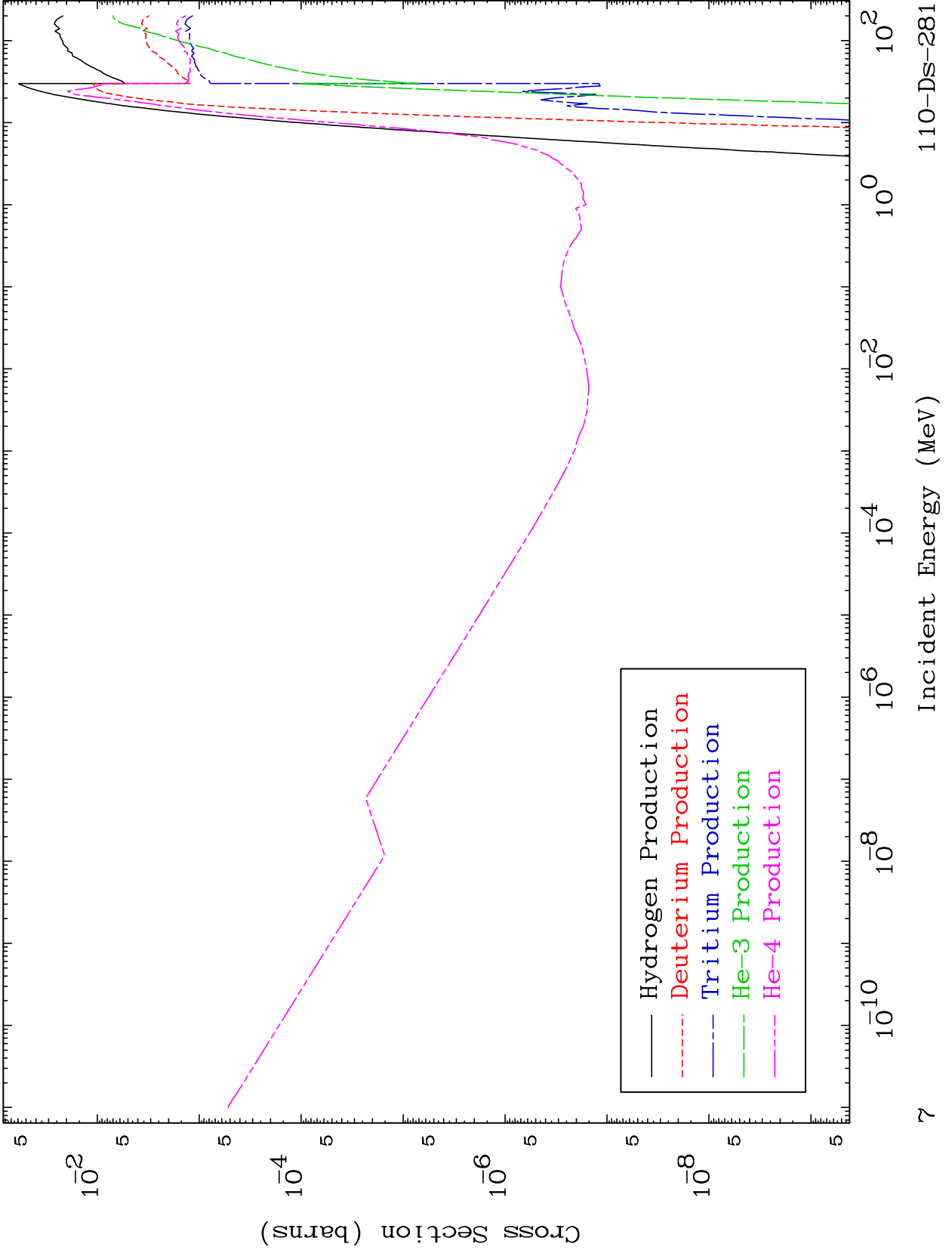
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Particle Production
293 Kelvin Cross Sections

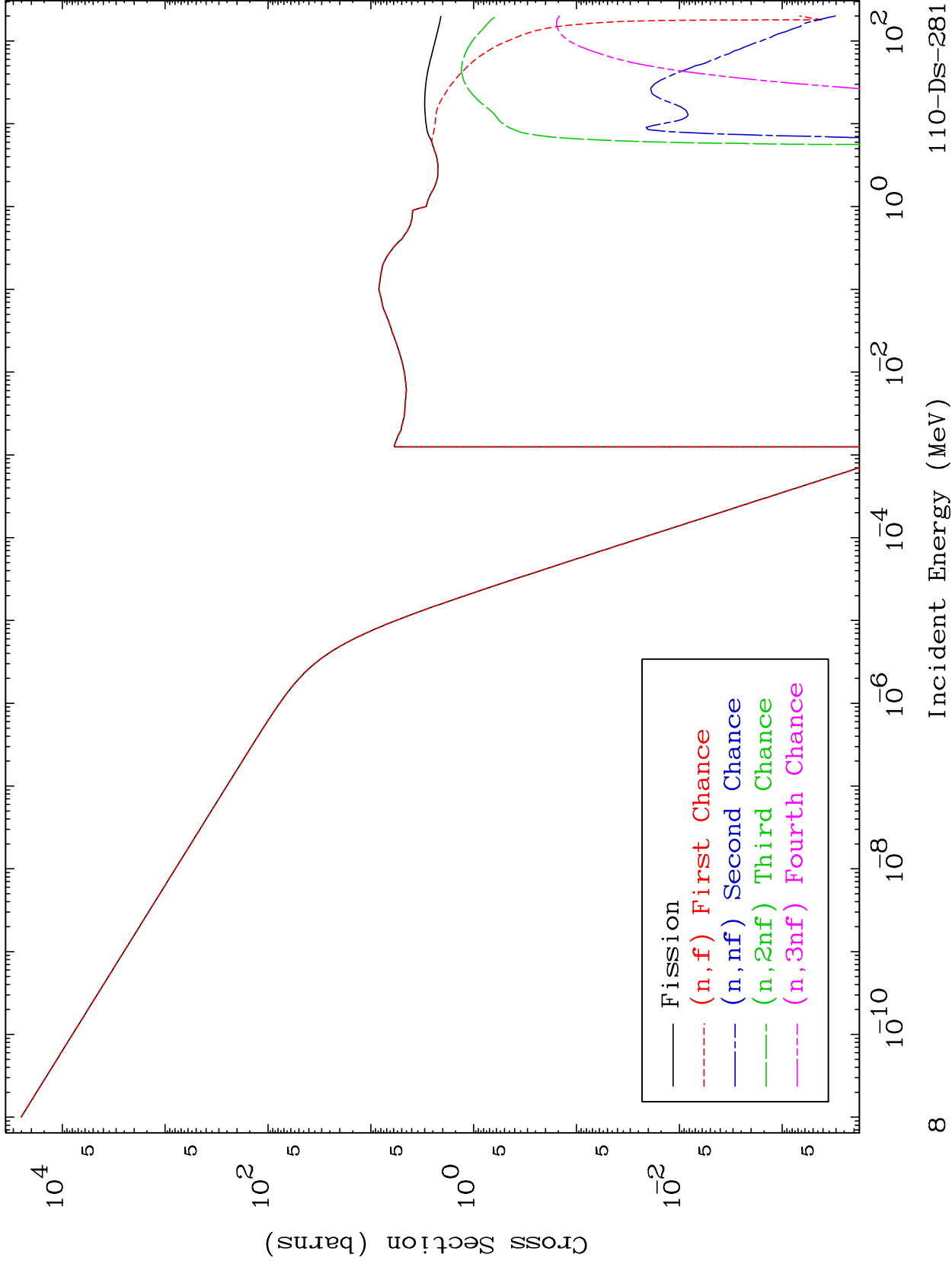
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Fission
293 Kelvin Cross Sections

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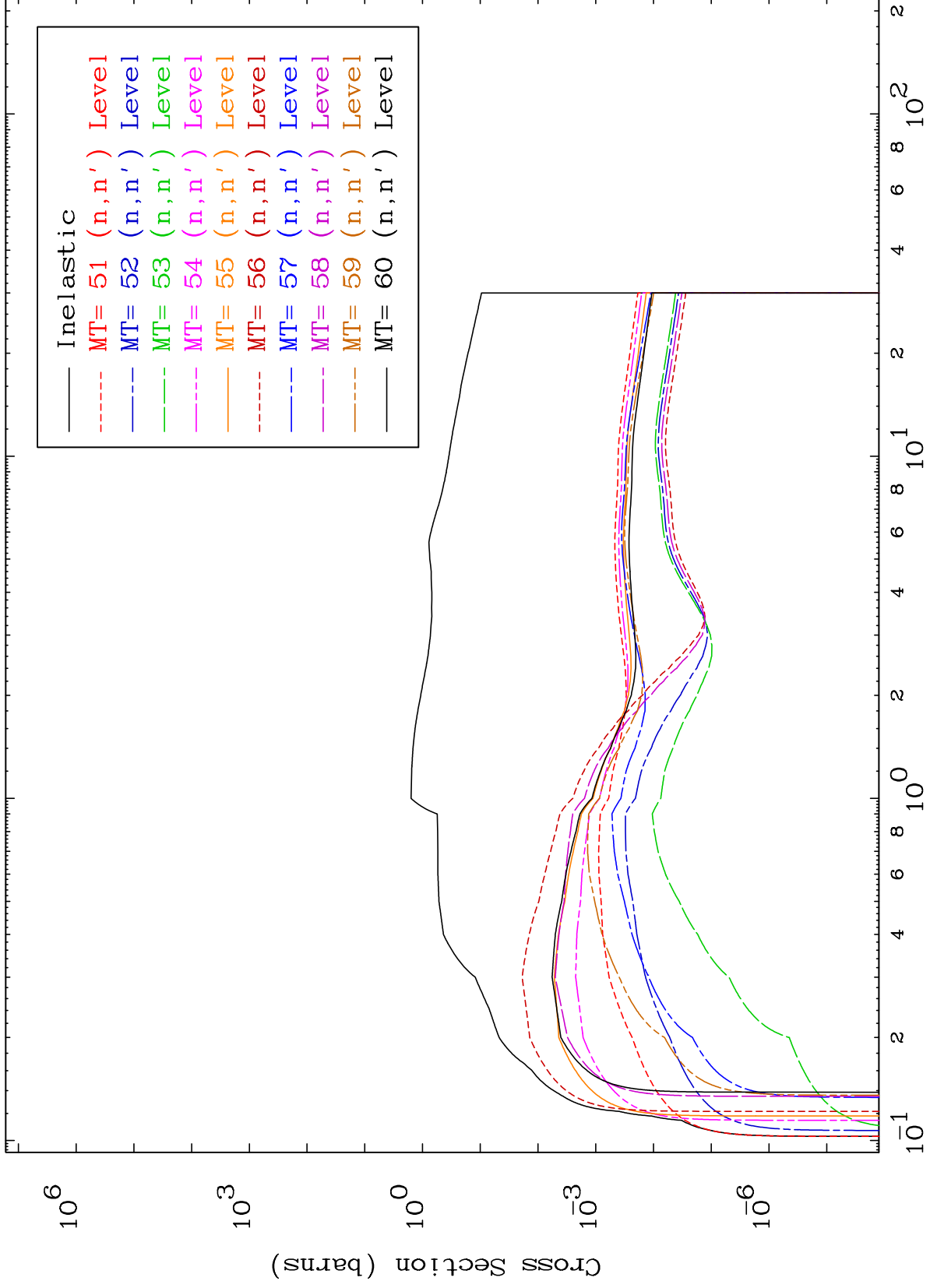


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(n,n') Levels

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293 Kelvin Cross Sections



Incident Energy (MeV)

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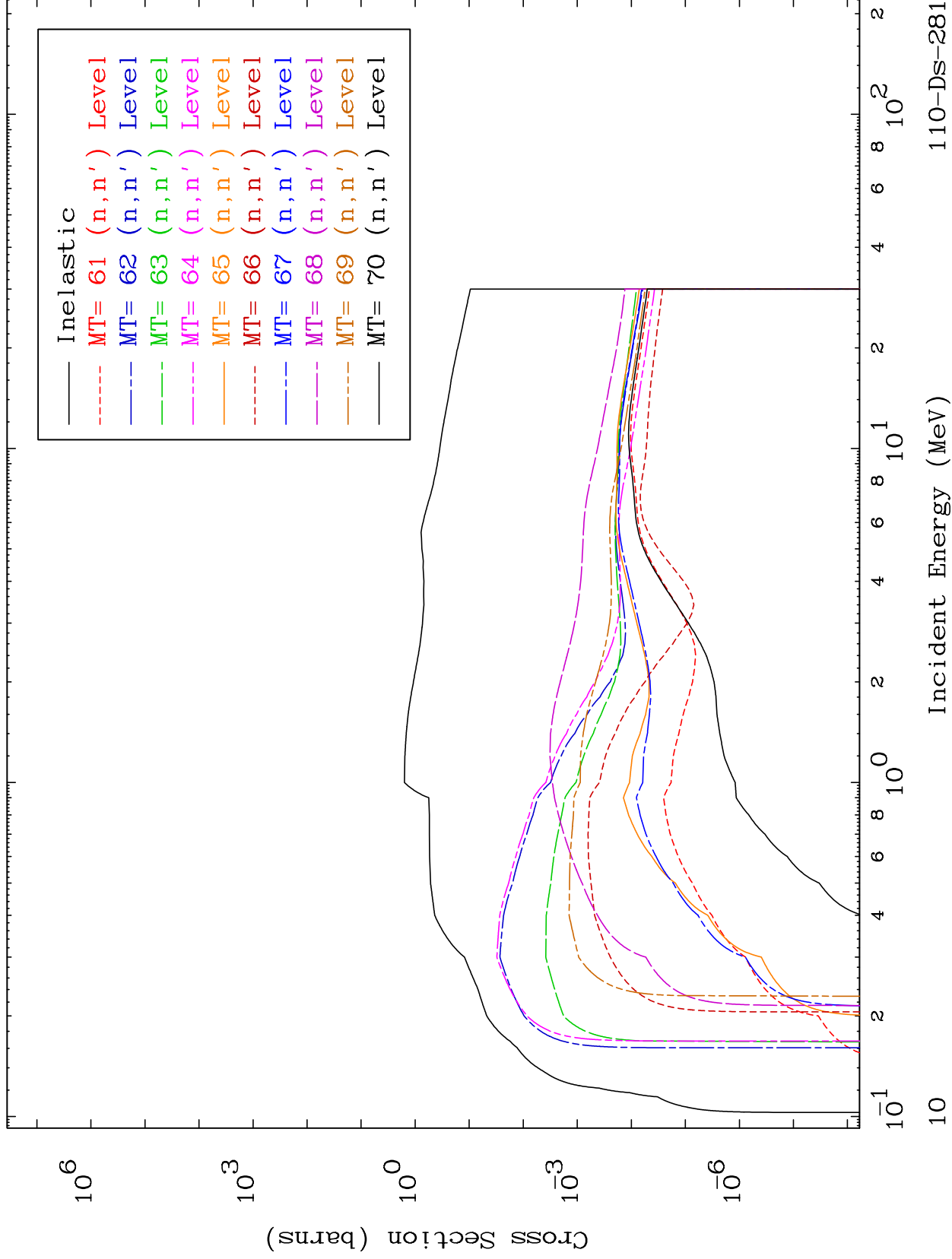
9

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(n,n') Levels

110-Ds-281

293 Kelvin Cross Sections



Incident Energy (MeV)

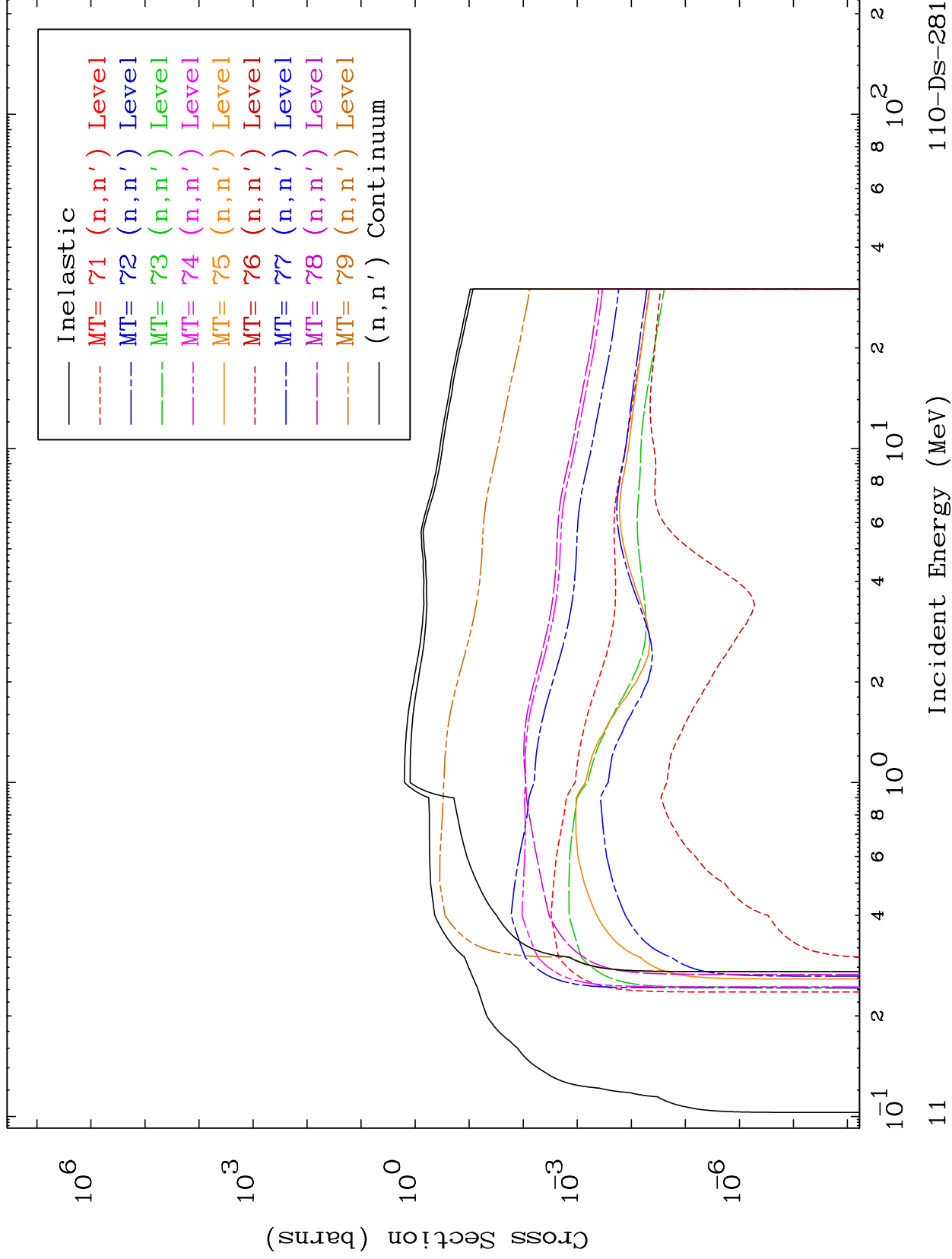
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(n,n') Levels

110-Ds-281

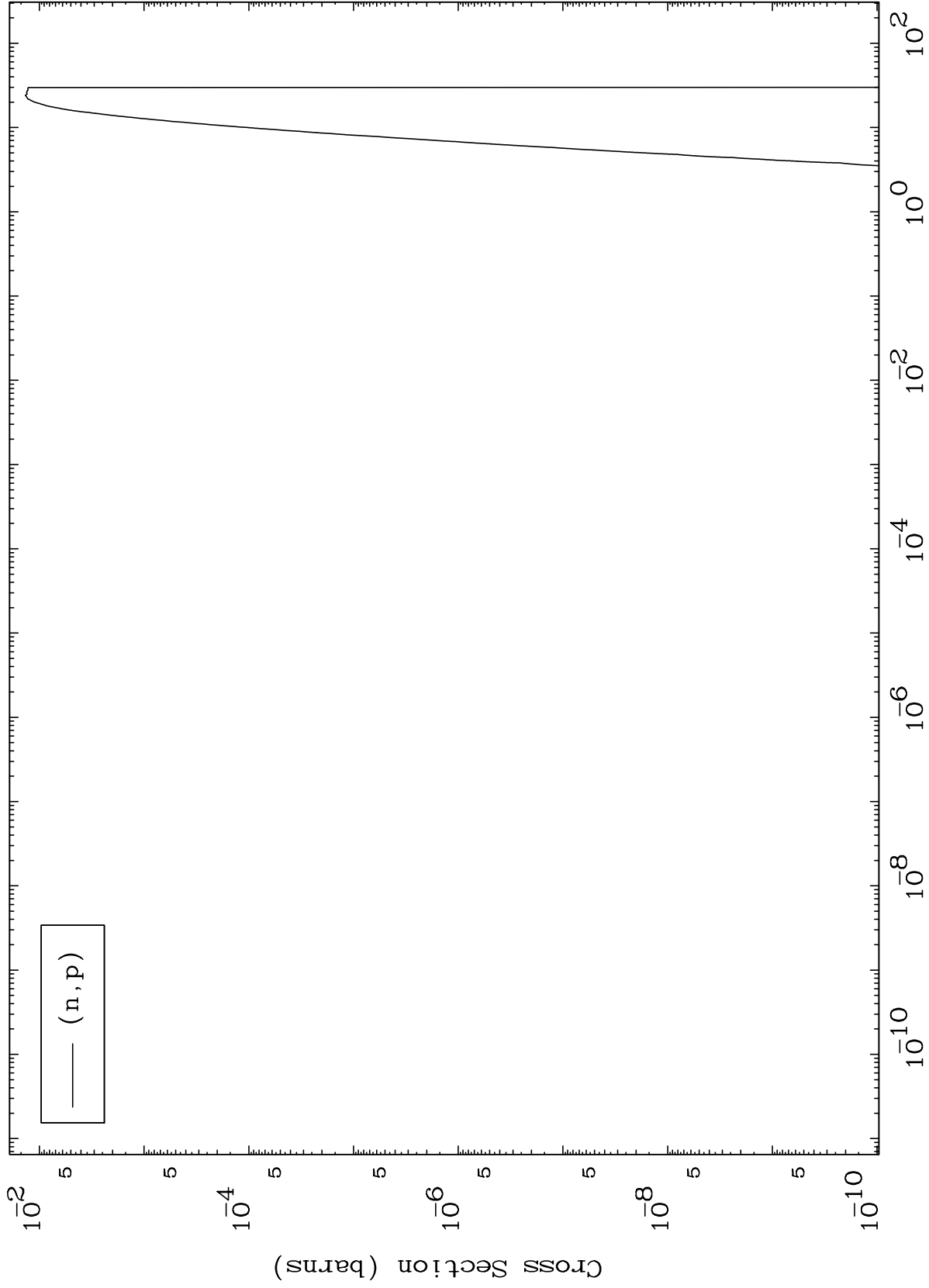
293 Kelvin Cross Sections



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(n,p) Levels
293 Kelvin Cross Sections

110-Ds-281



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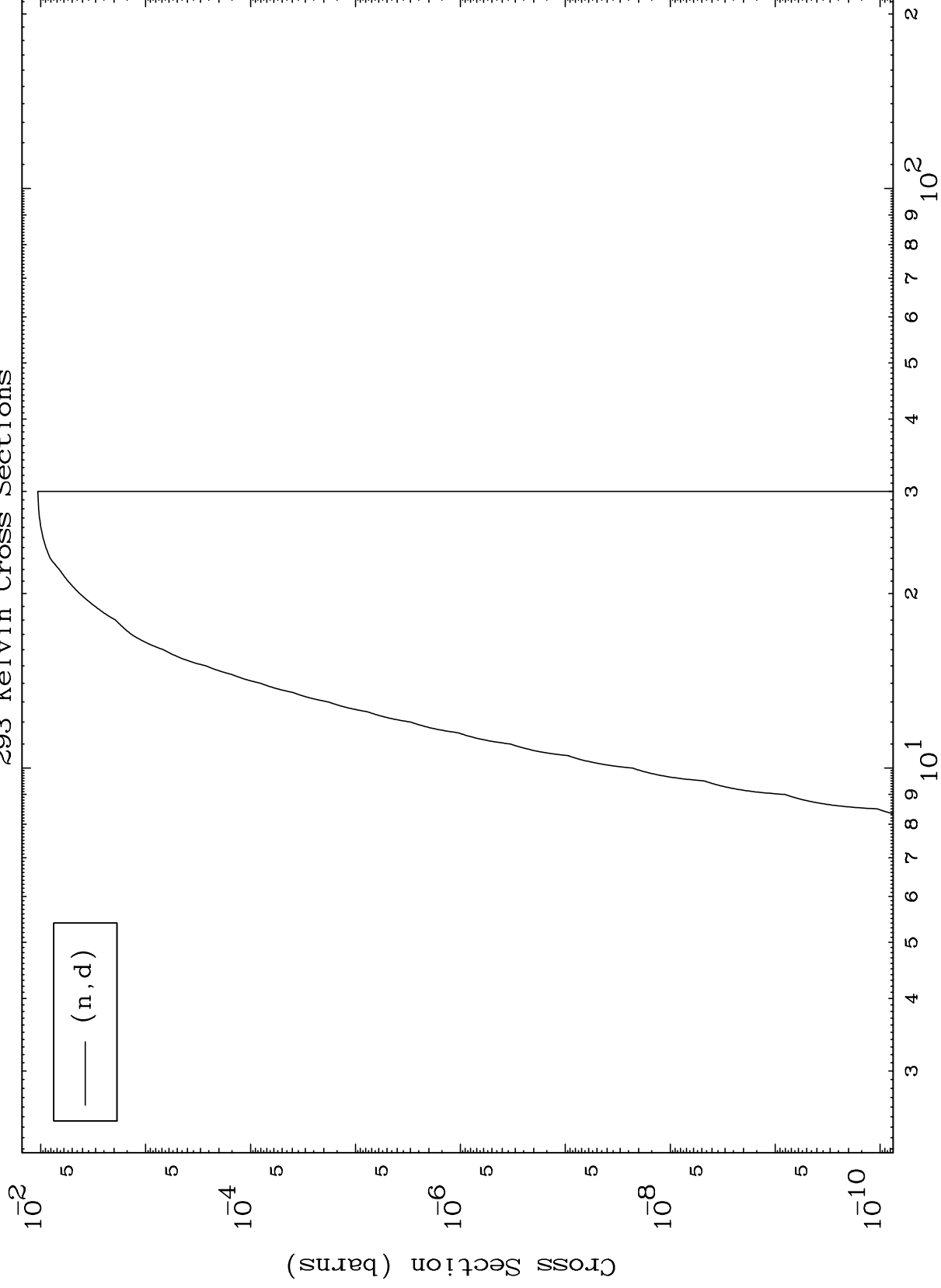
Incident Energy (MeV)

110-Ds-281

MAT 1081

(n,d) Levels
293 Kelvin Cross Sections

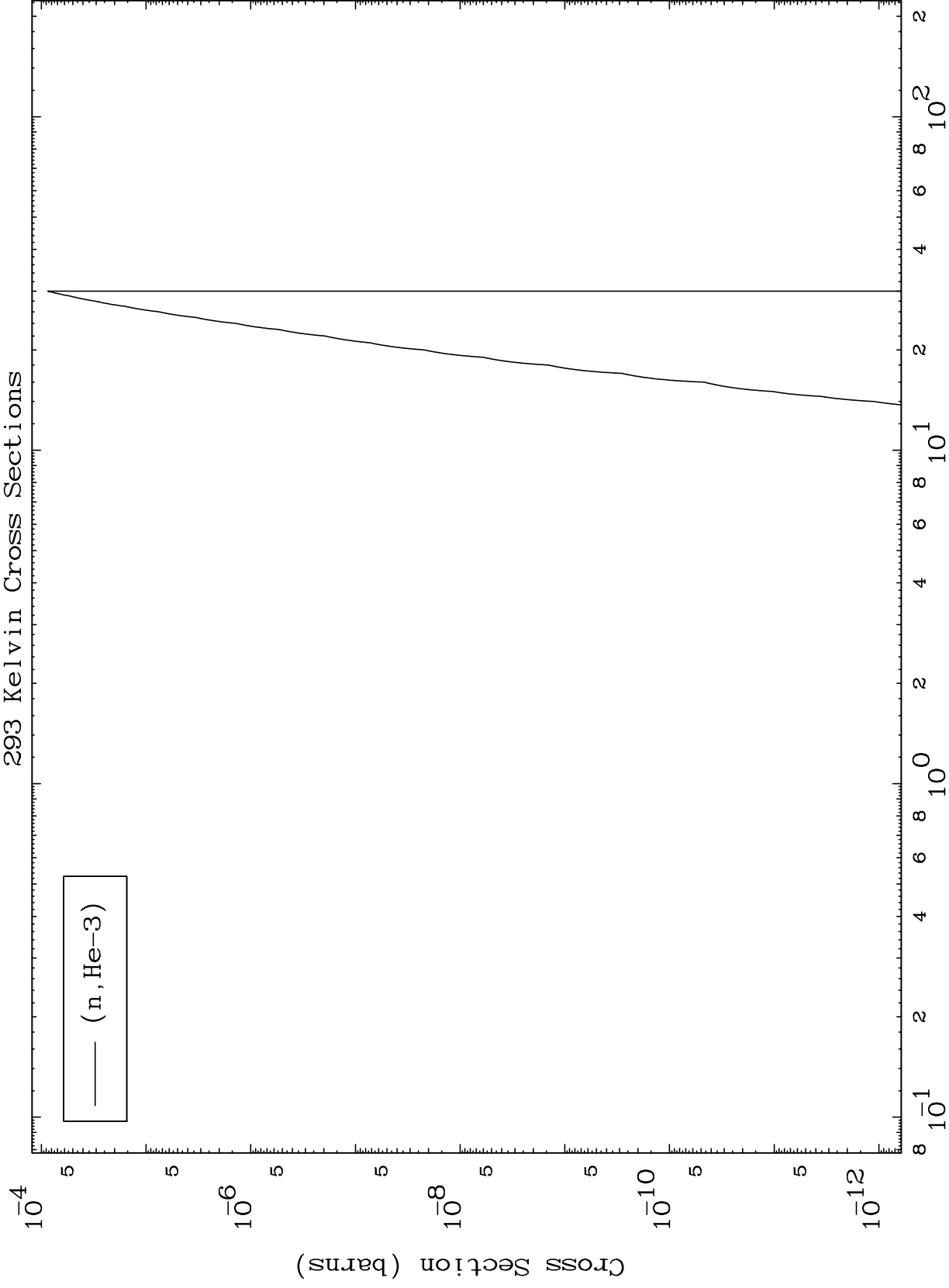
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(n,He3) Levels
293 Kelvin Cross Sections

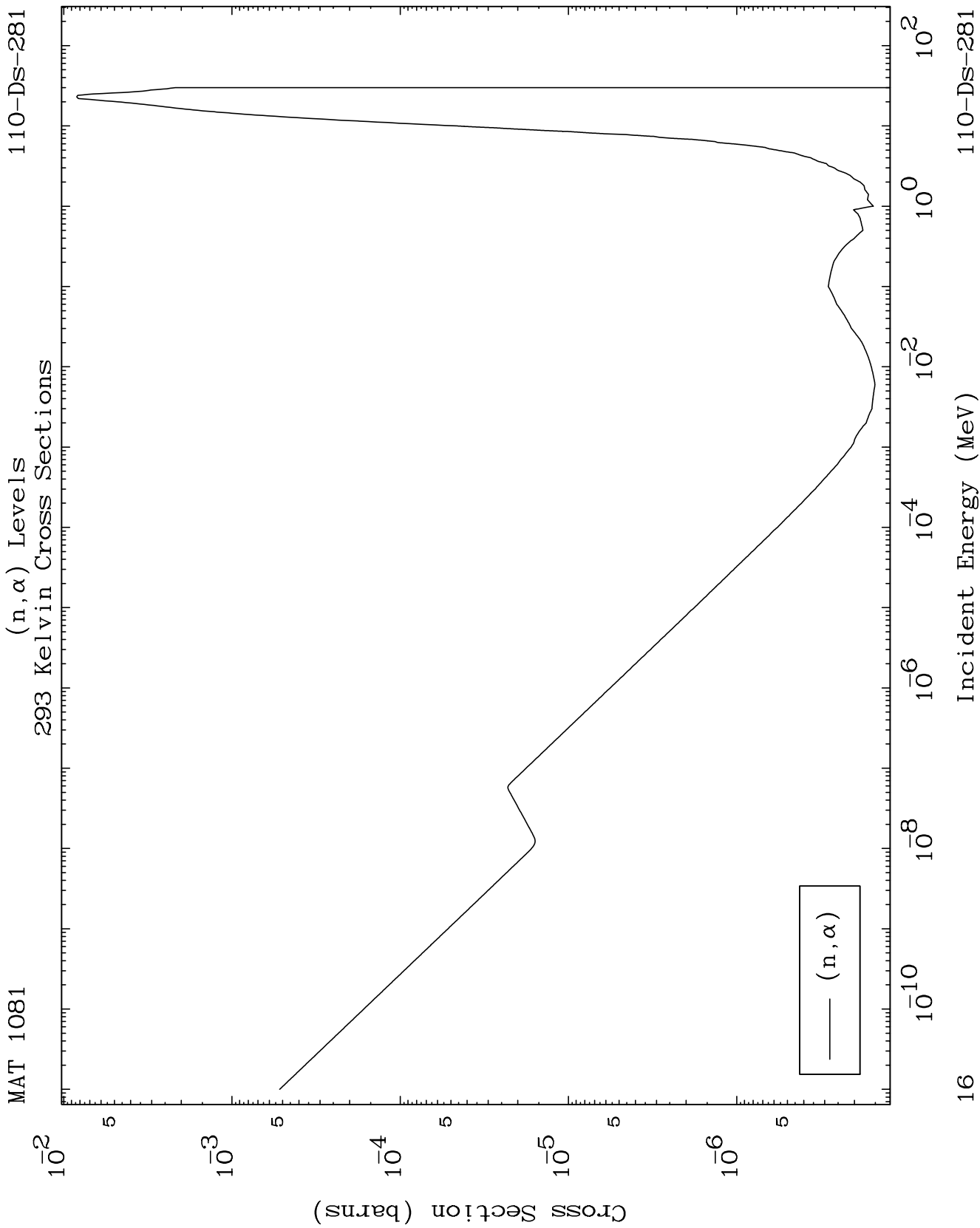
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Incident Energy (MeV)

110-Ds-281

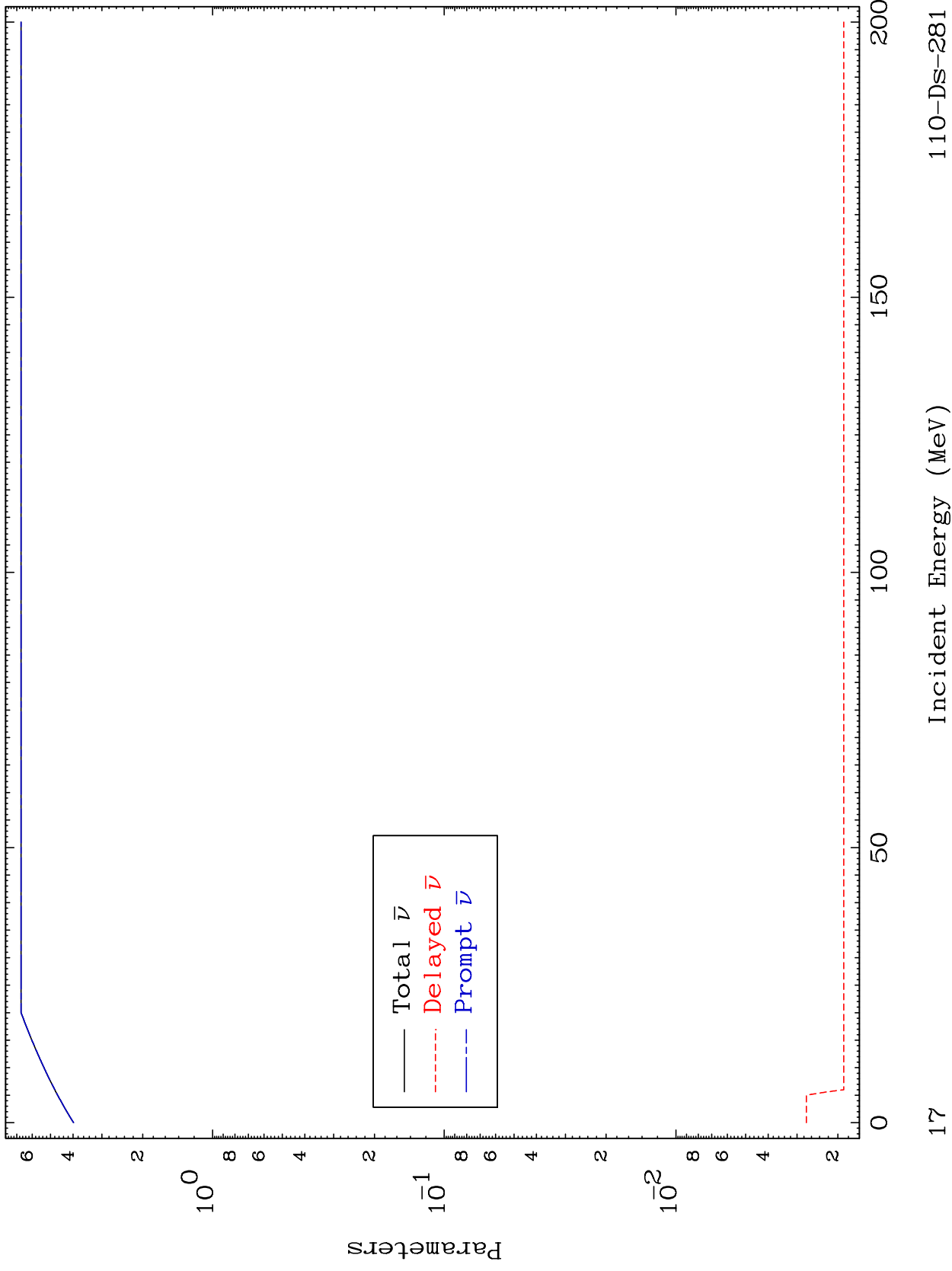
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Energy Release
Parameters

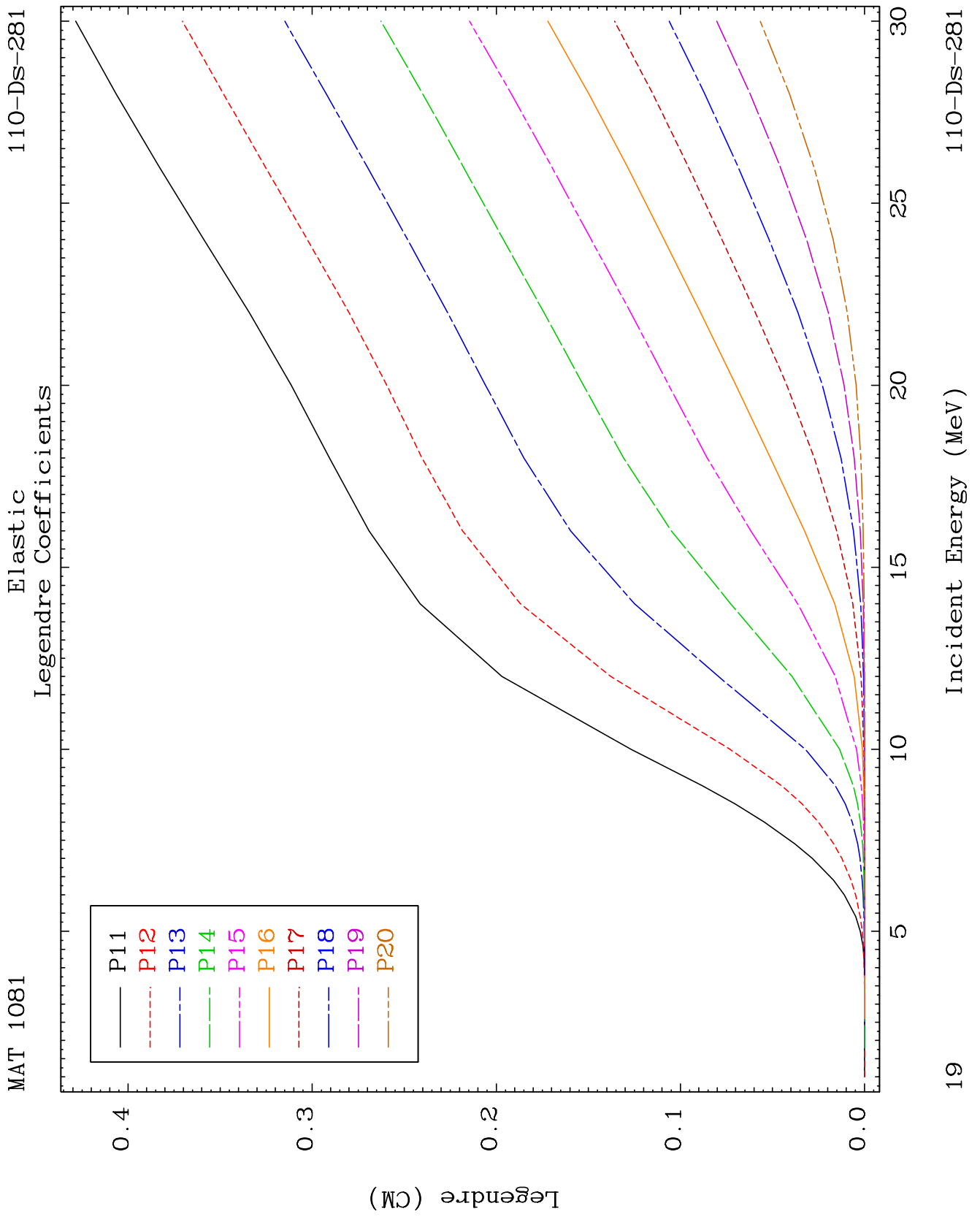
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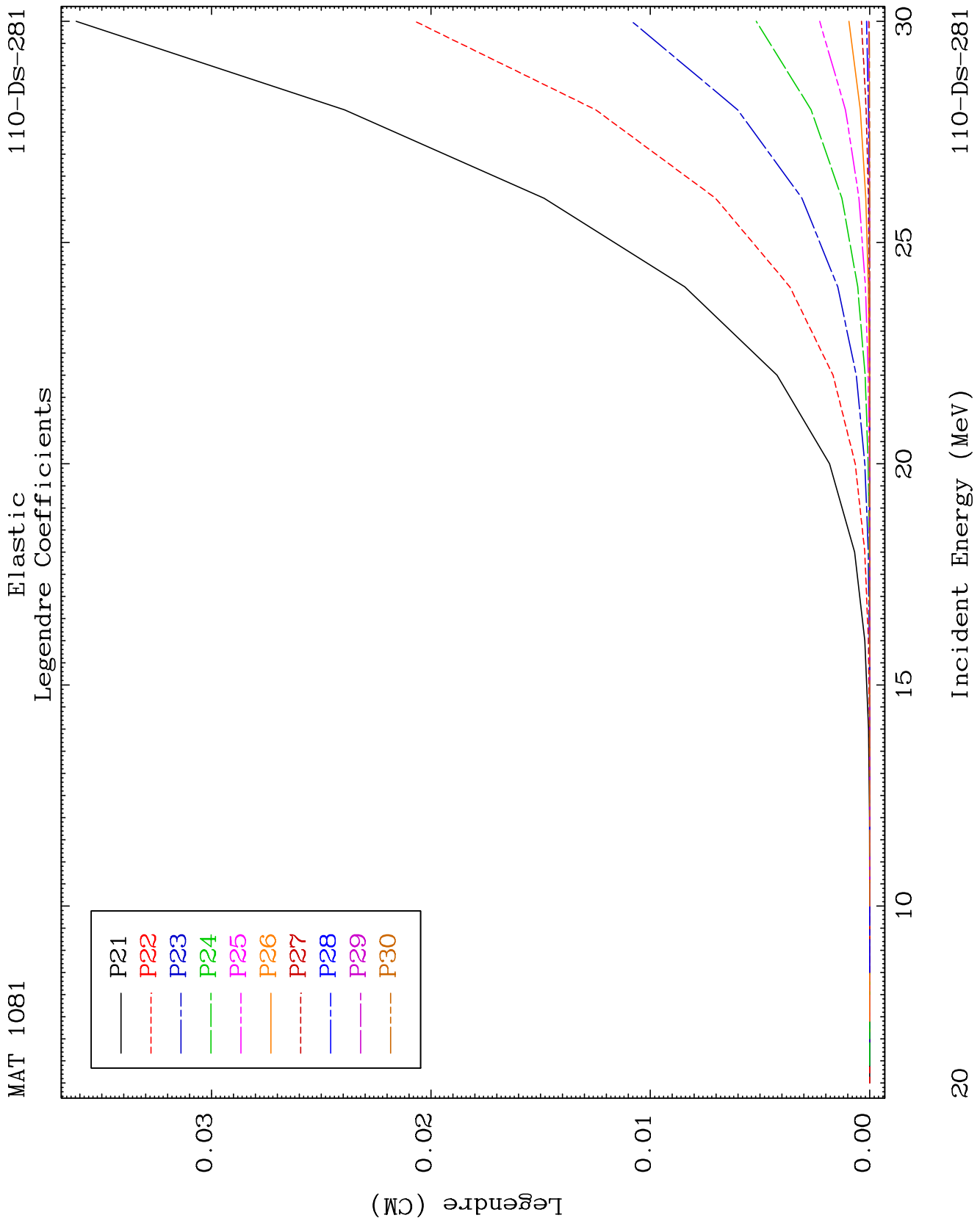


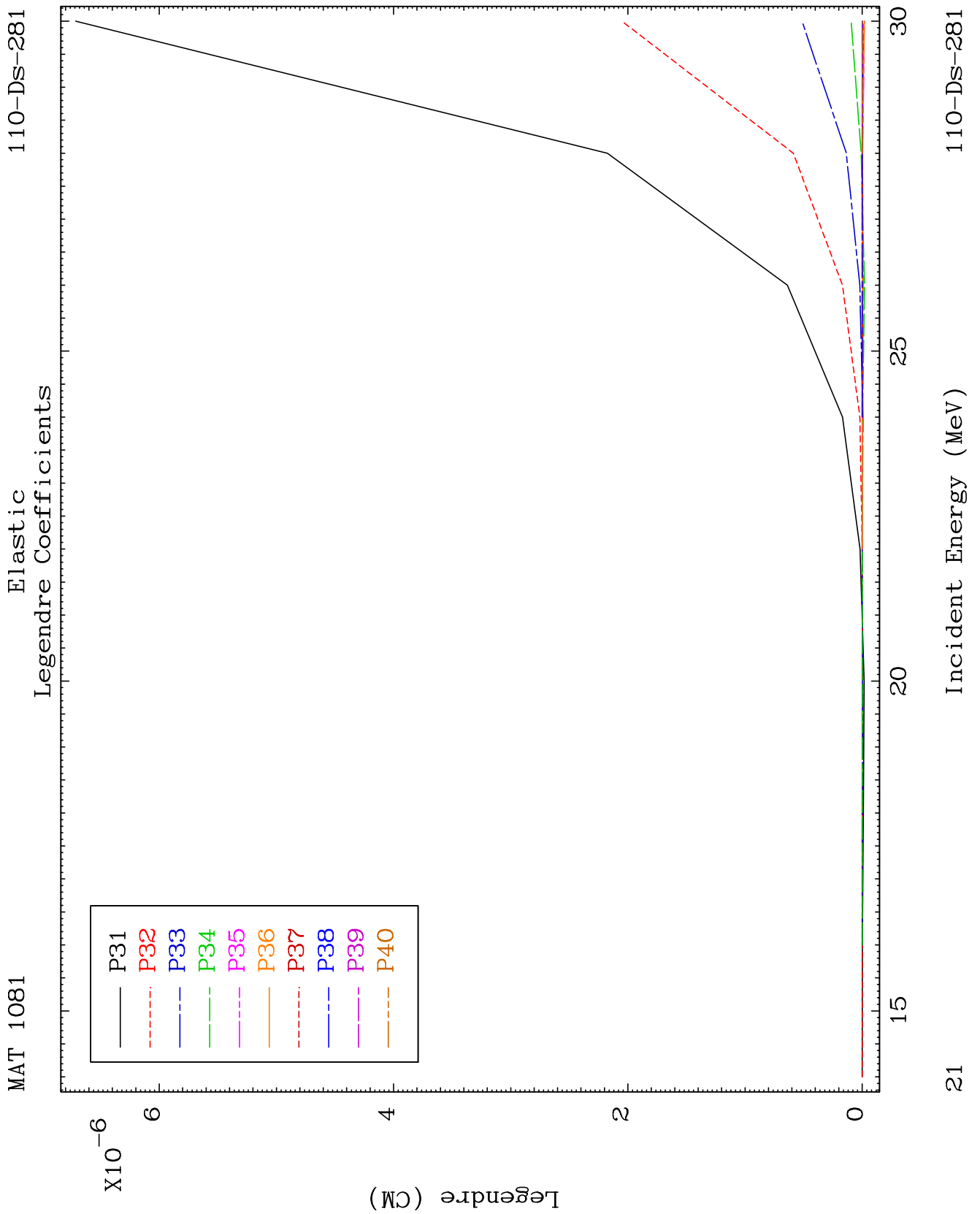
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Incident Energy (MeV)

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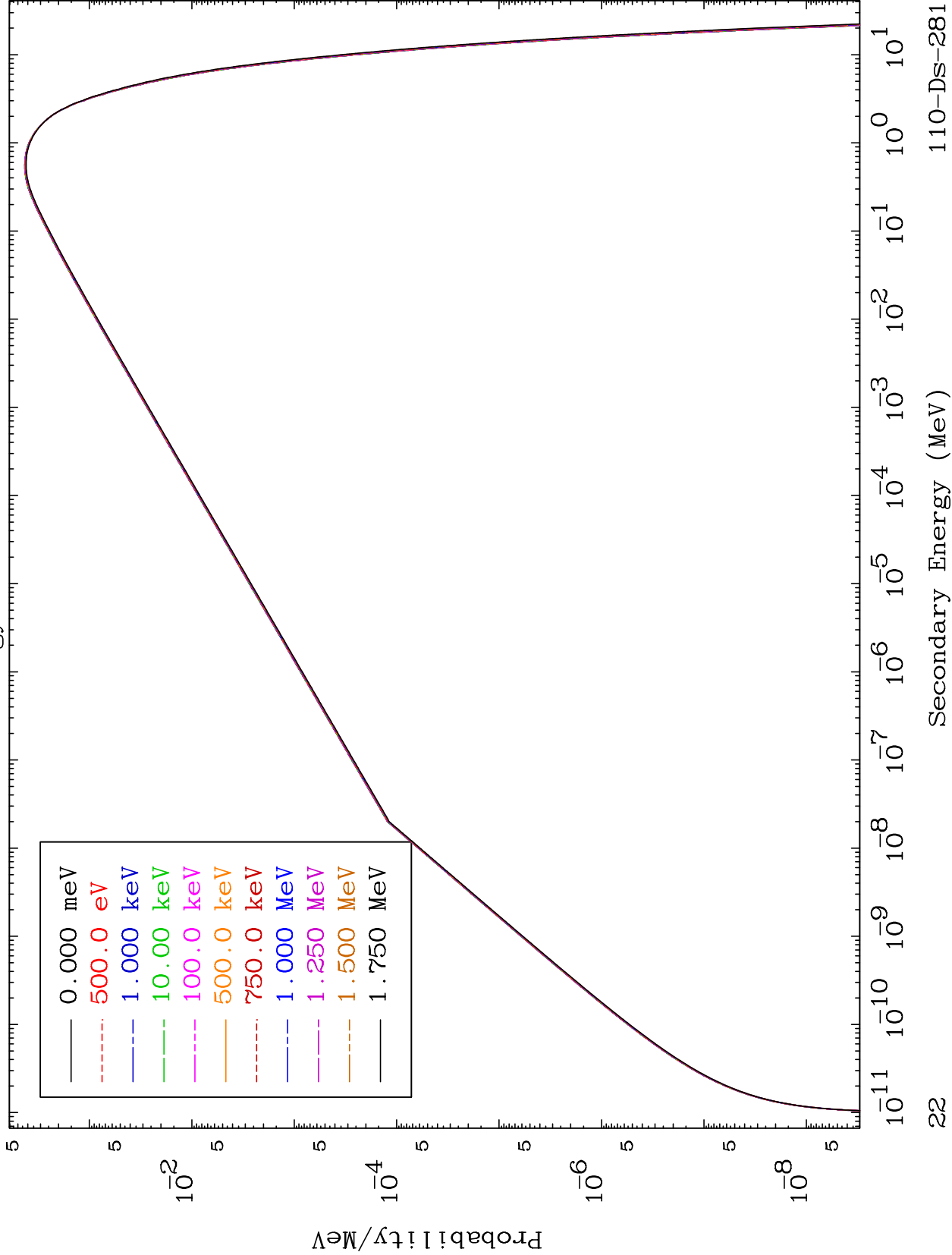




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Fission Energy Distributions

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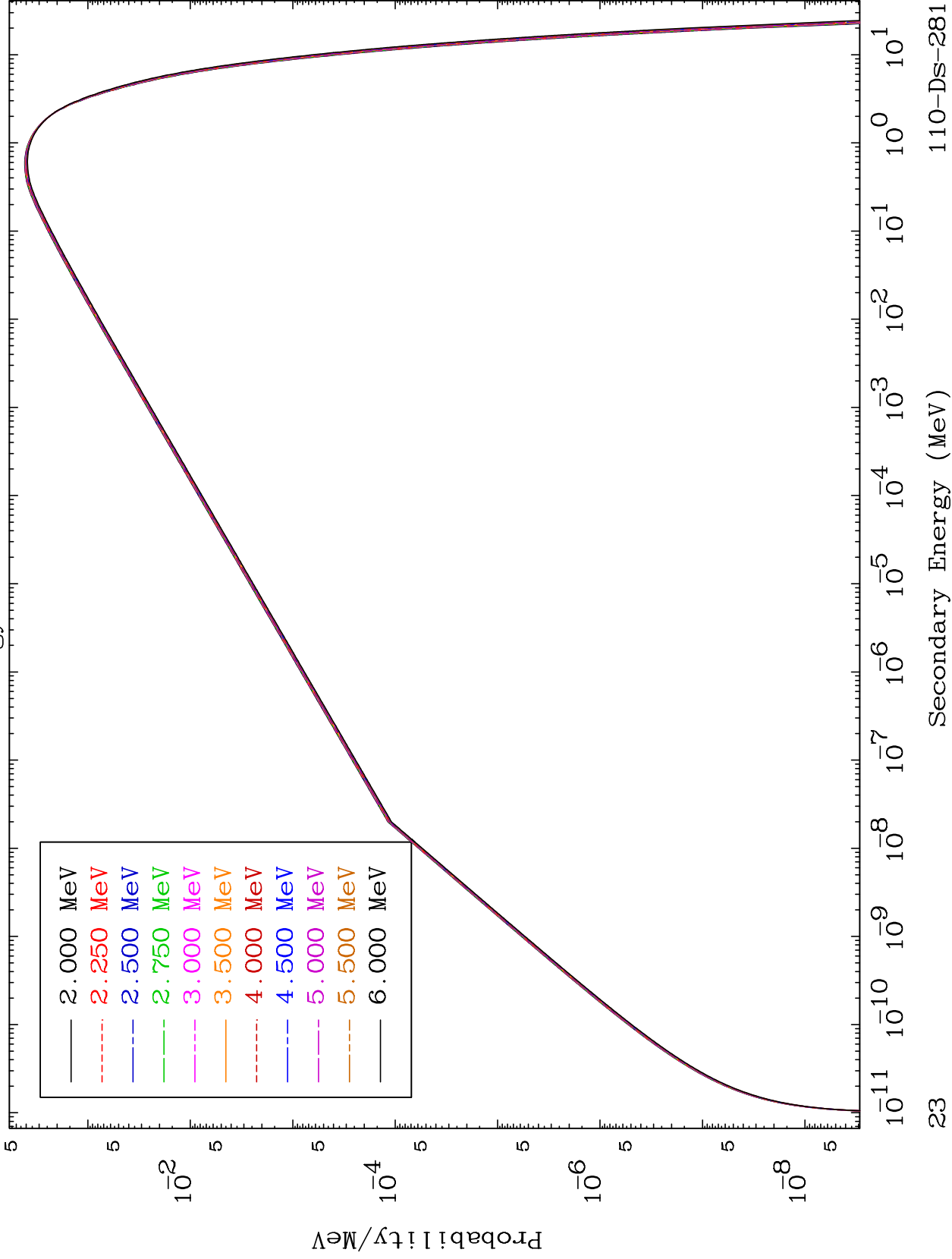
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Secondary Energy (MeV)

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Fission Energy Distributions

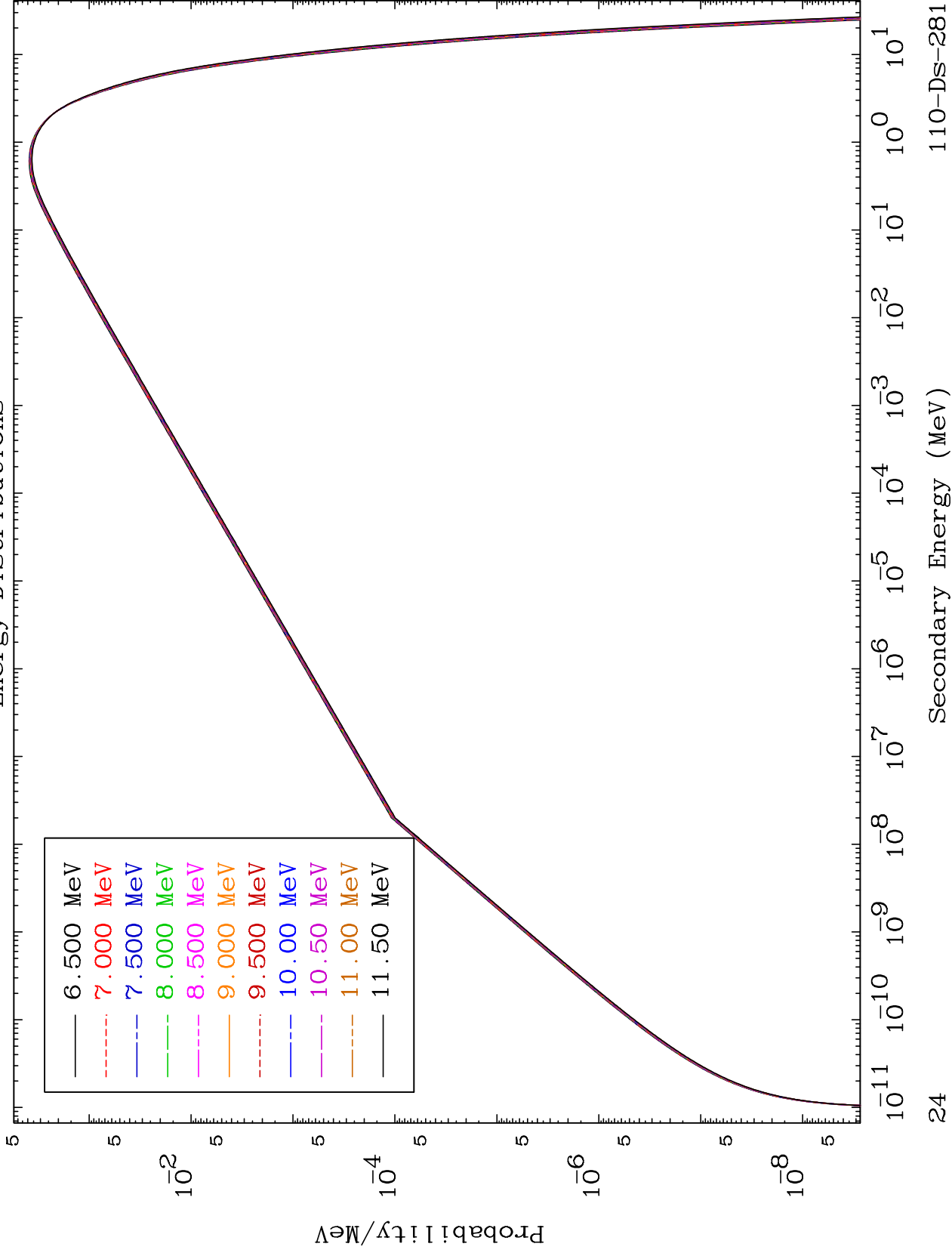
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Fission Energy Distributions

110-Ds-281



110-Ds-281

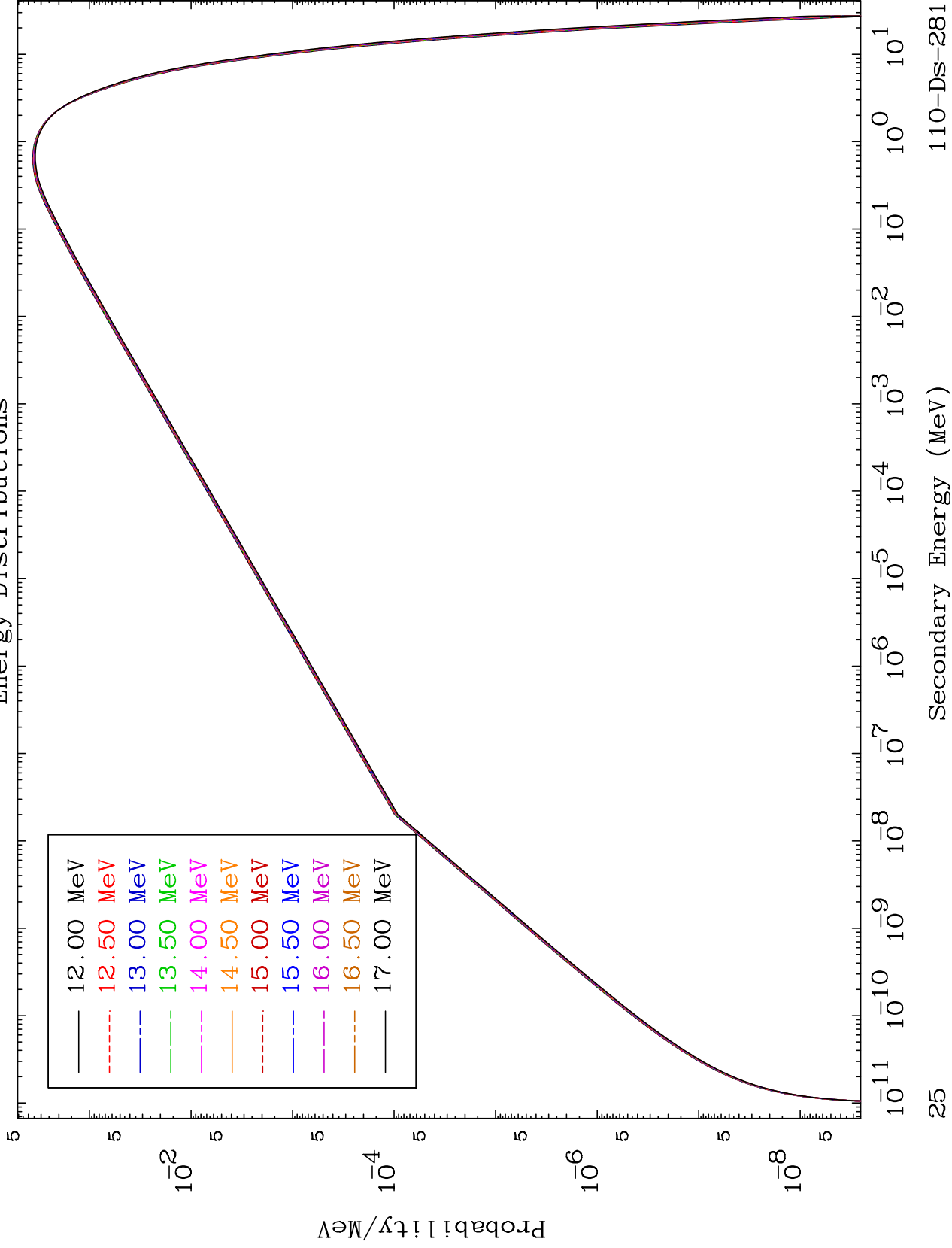
Secondary Energy (MeV)

24

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Fission
Energy Distributions

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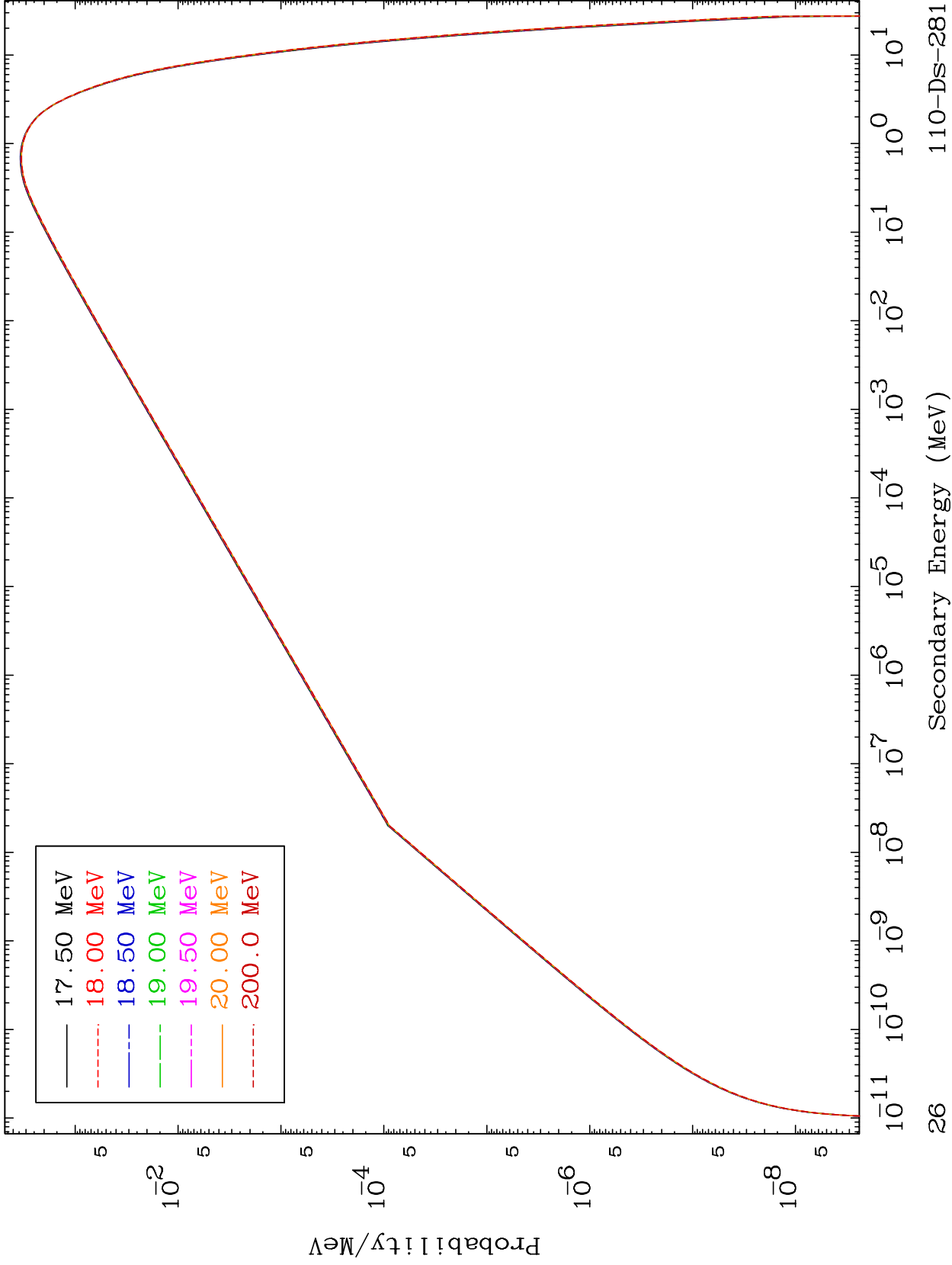


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Fission Energy Distributions

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$(n, n') \alpha$

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Radionuclide Production Cross Section

