

Program EVALPLOT
(Version 2021-1)

by

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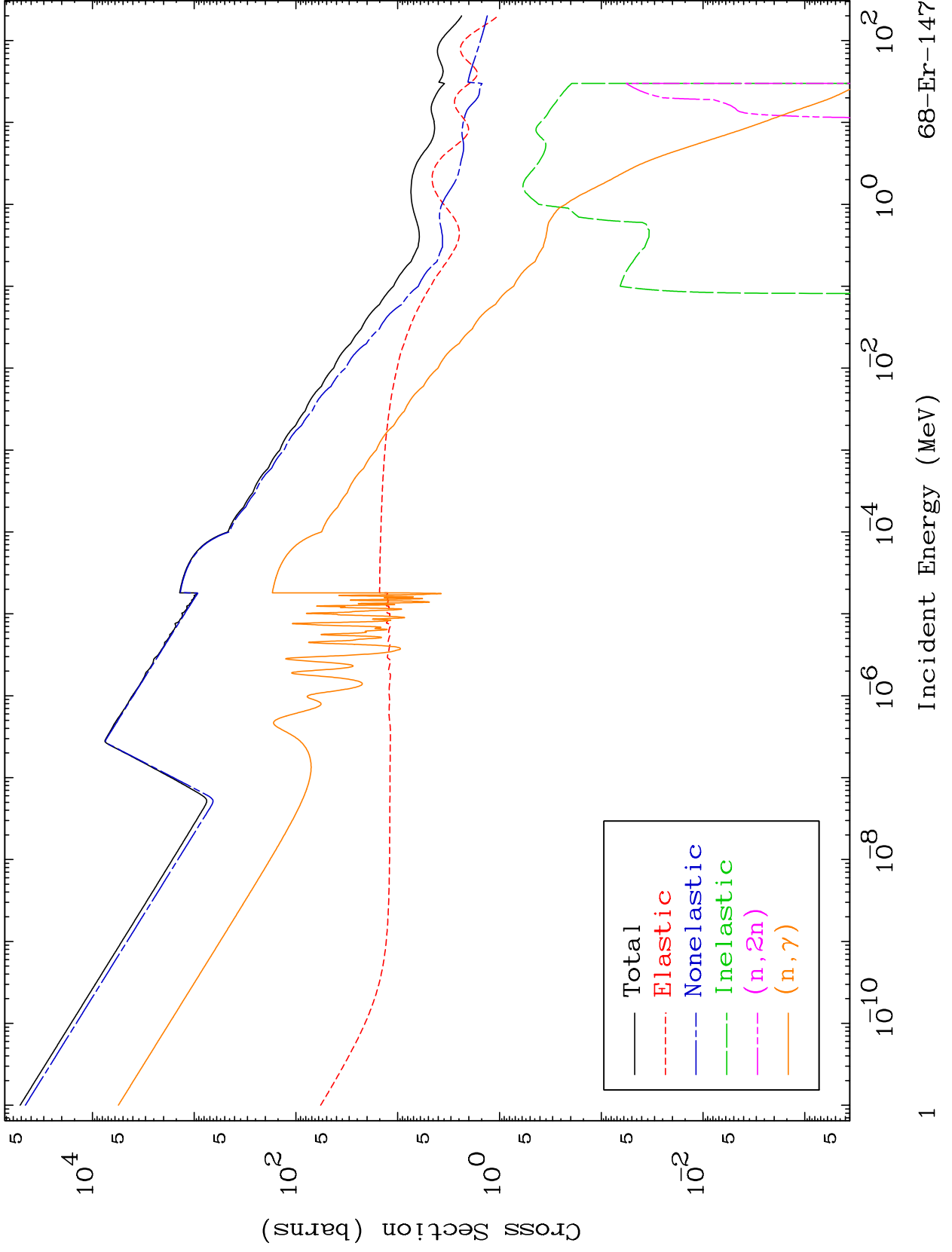
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Press Mouse Button to Start

MAT 6780

Neutron Major
293 Kelvin Cross Sections

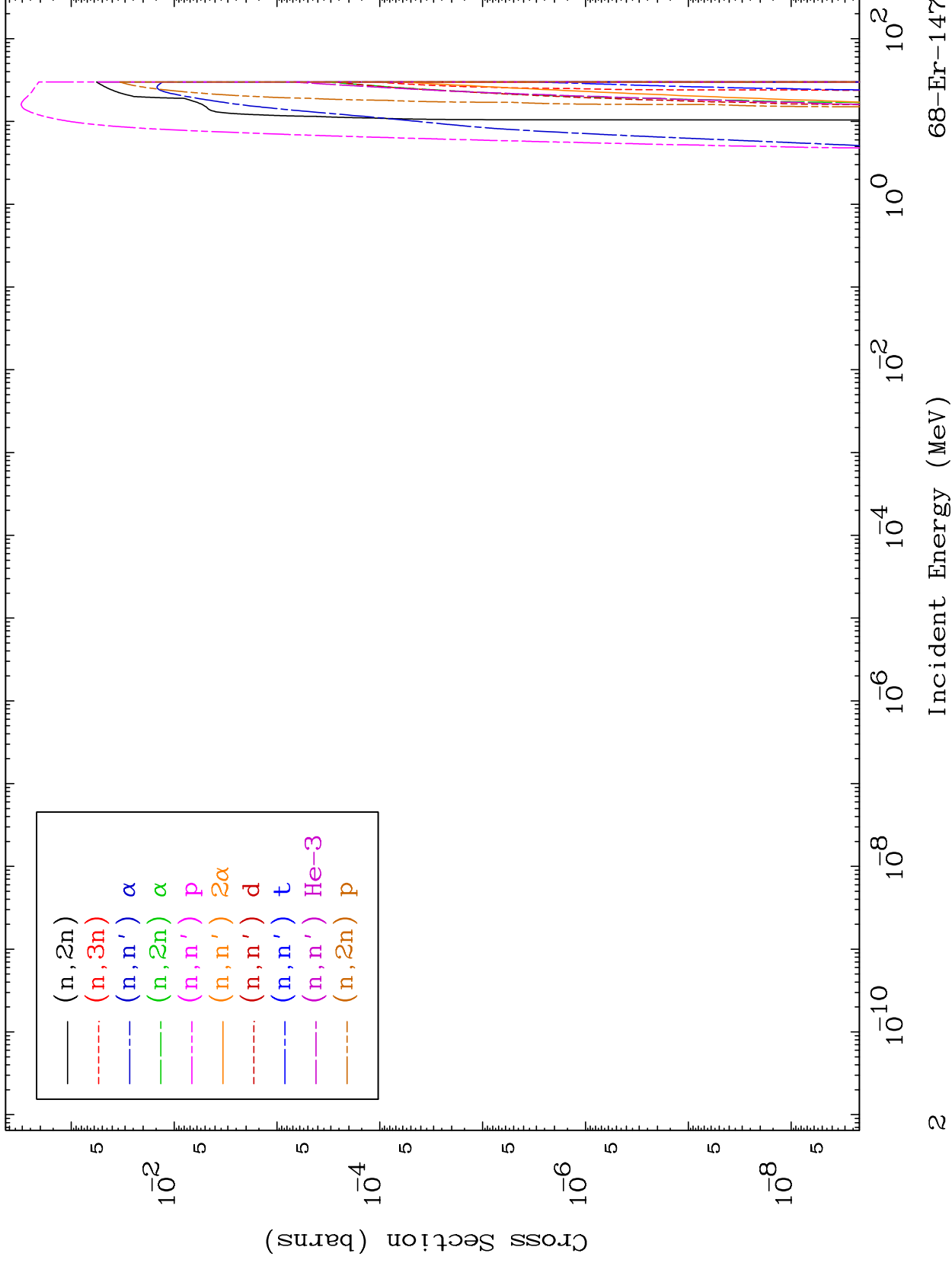
68-Er-147



MAT 6780

Neutron Absorption
293 Kelvin Cross Sections

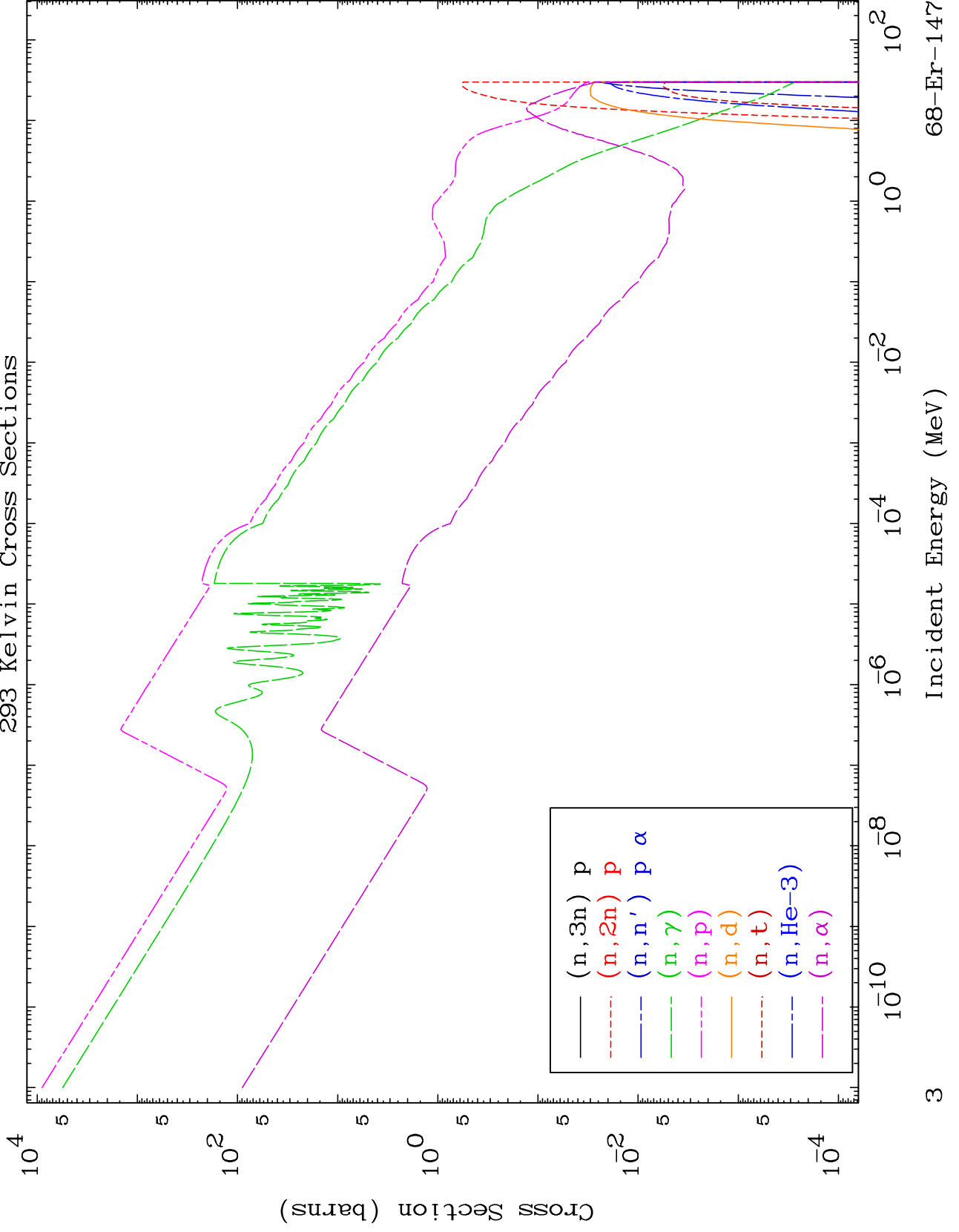
68-Er-147

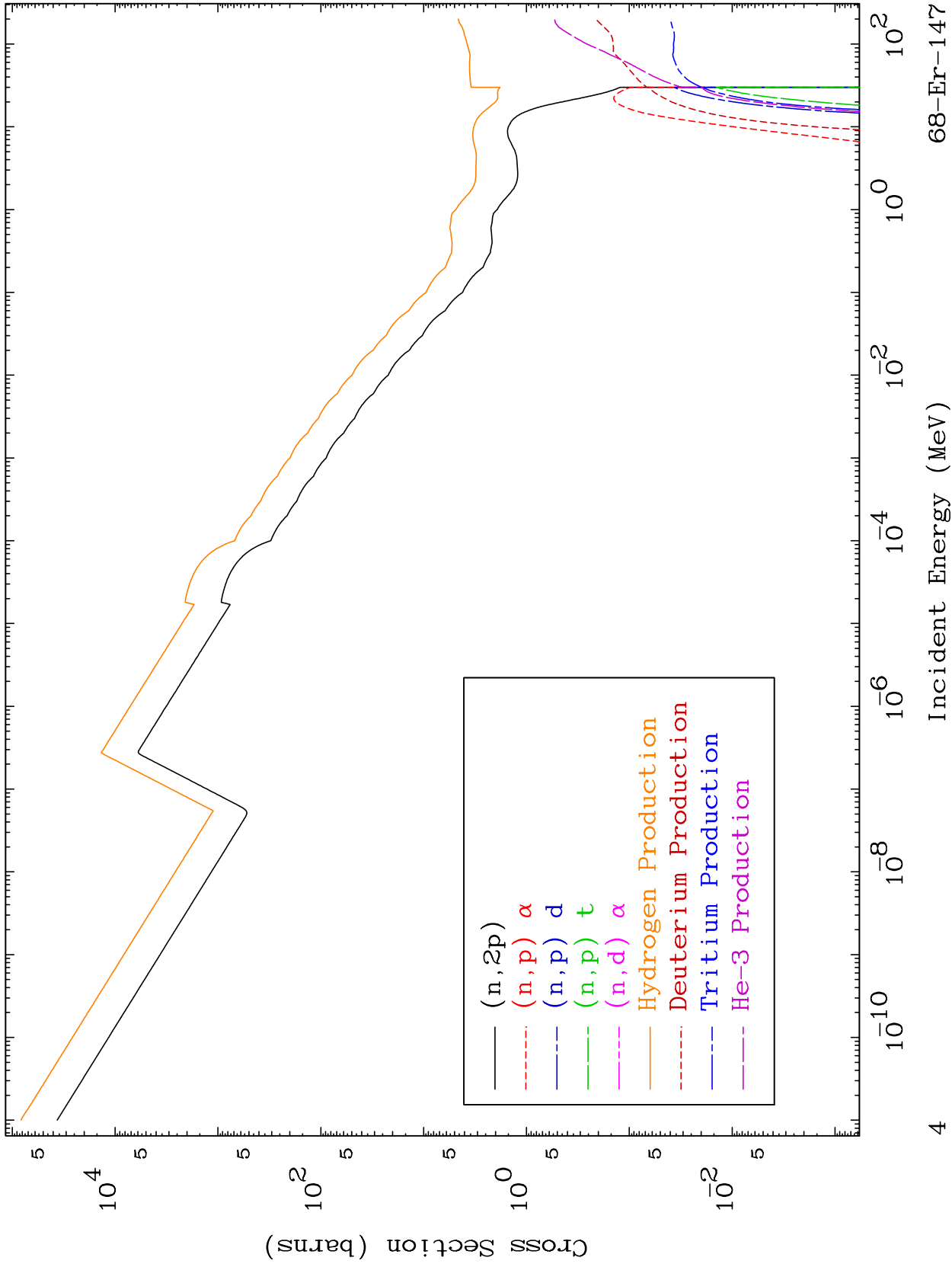


MAT 6780

Neutron Absorption
293 Kelvin Cross Sections

68-Er-147

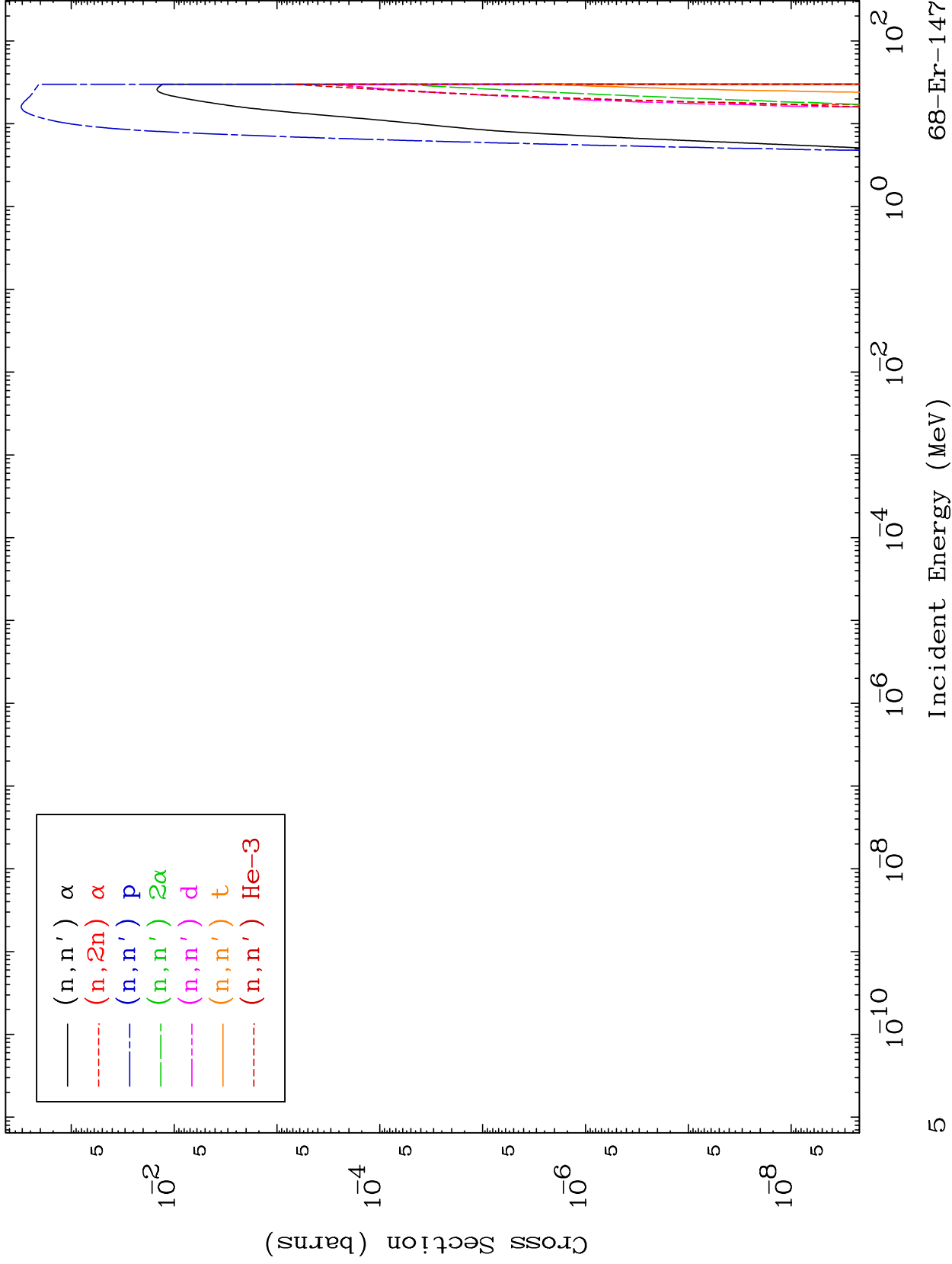




MAT 6780

Charged Particle
293 Kelvin Cross Sections

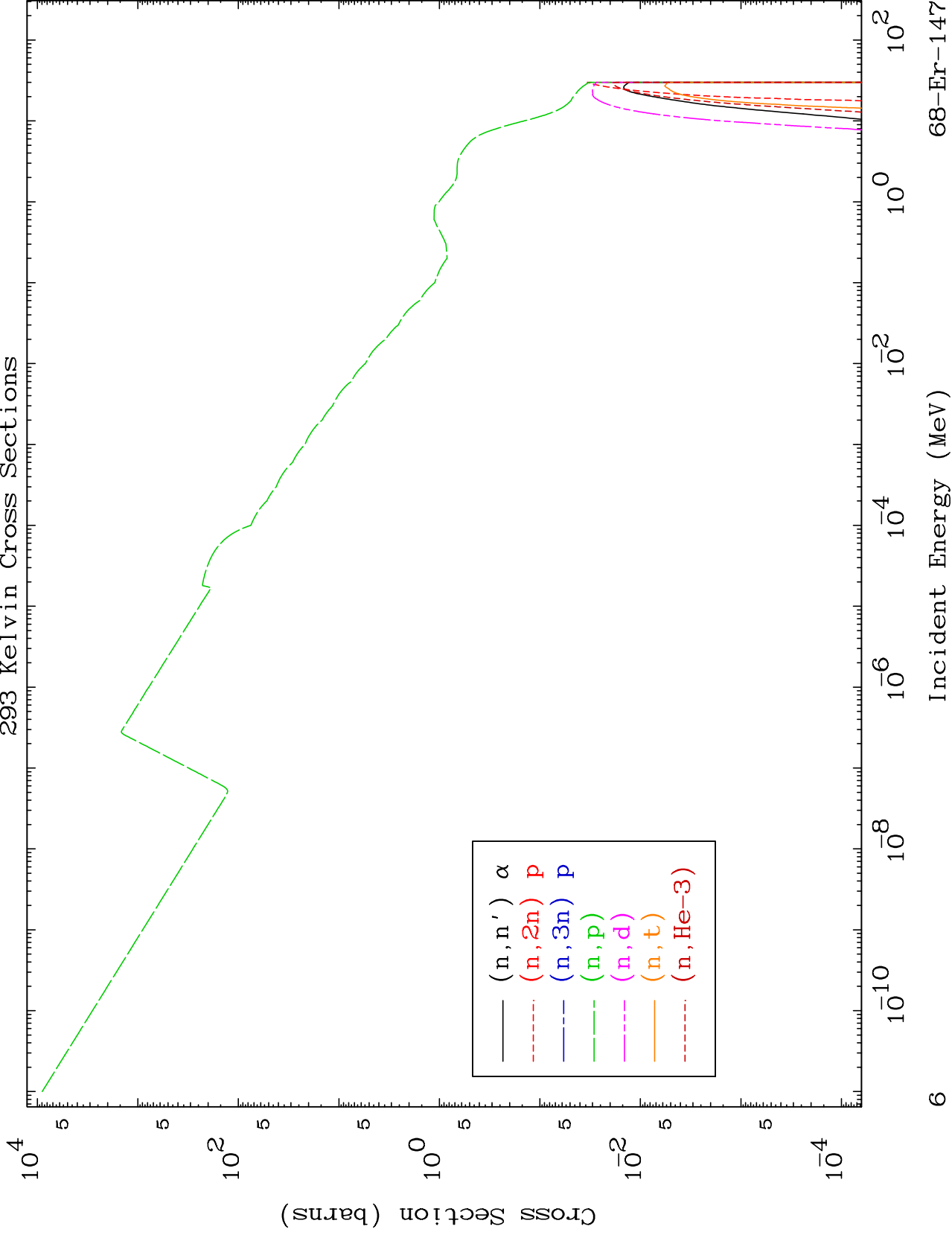
68-Er-147



MAT 6780

Charged Particle
293 Kelvin Cross Sections

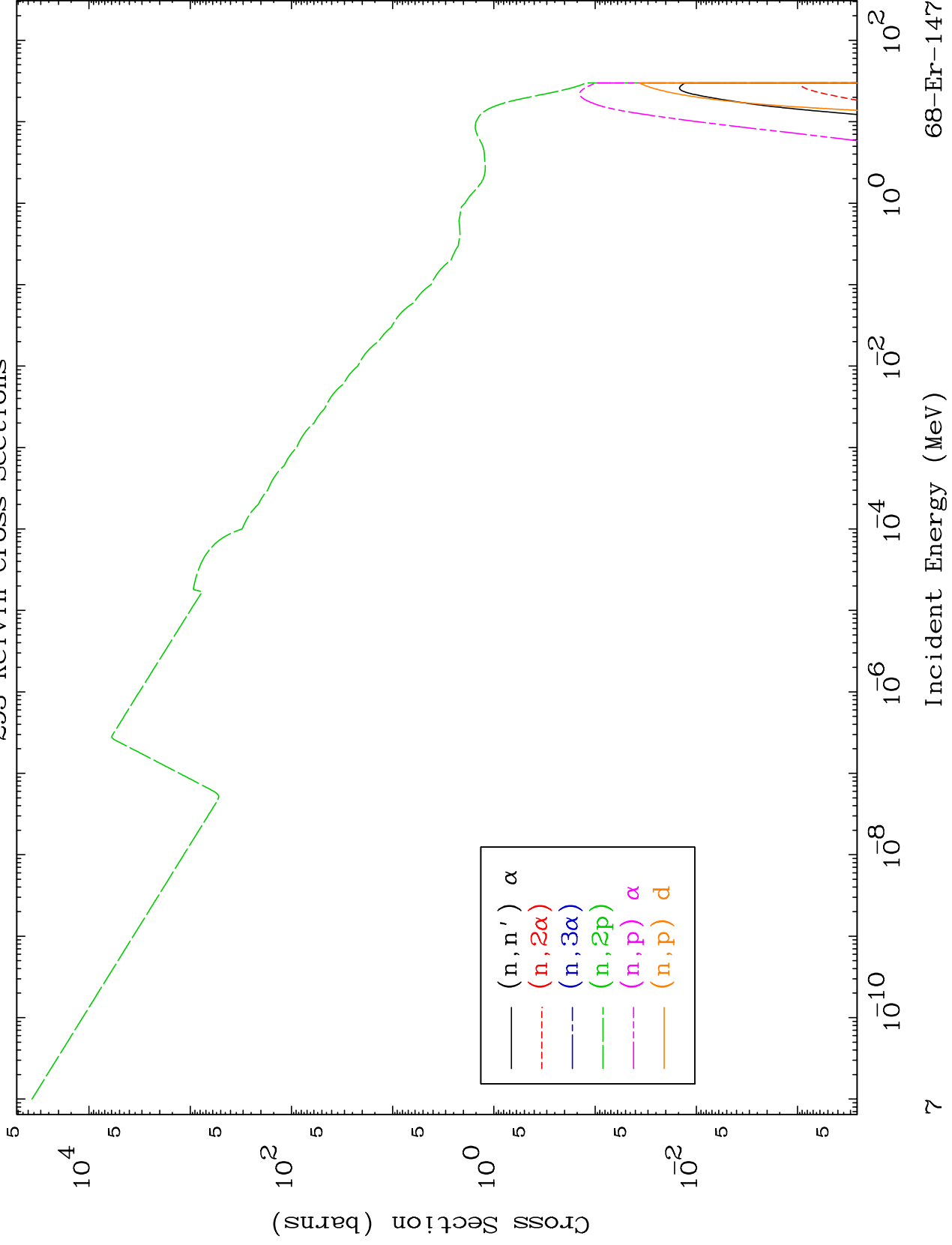
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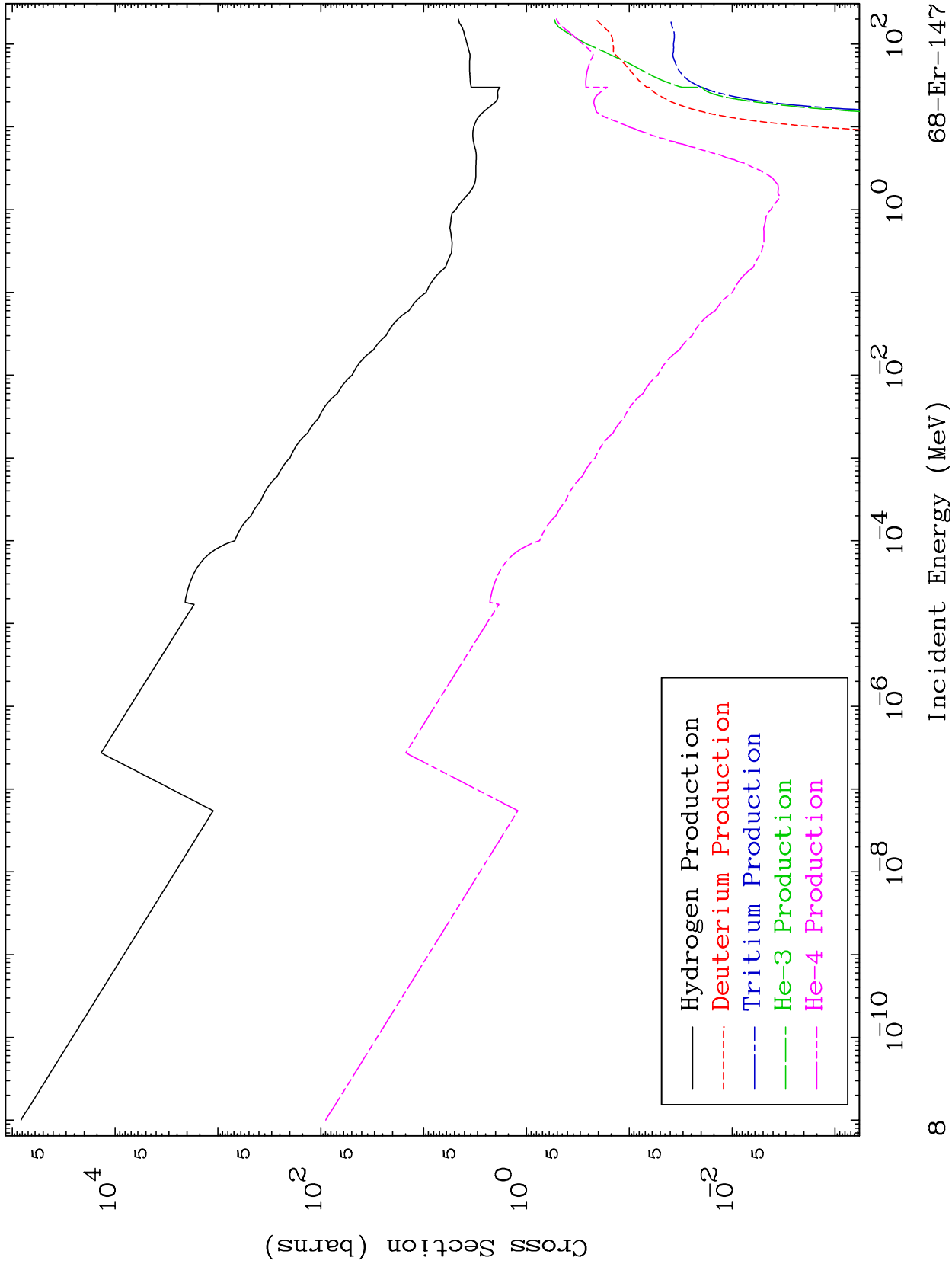


MAT 6780

Charged Particle
293 Kelvin Cross Sections

68-Er-147



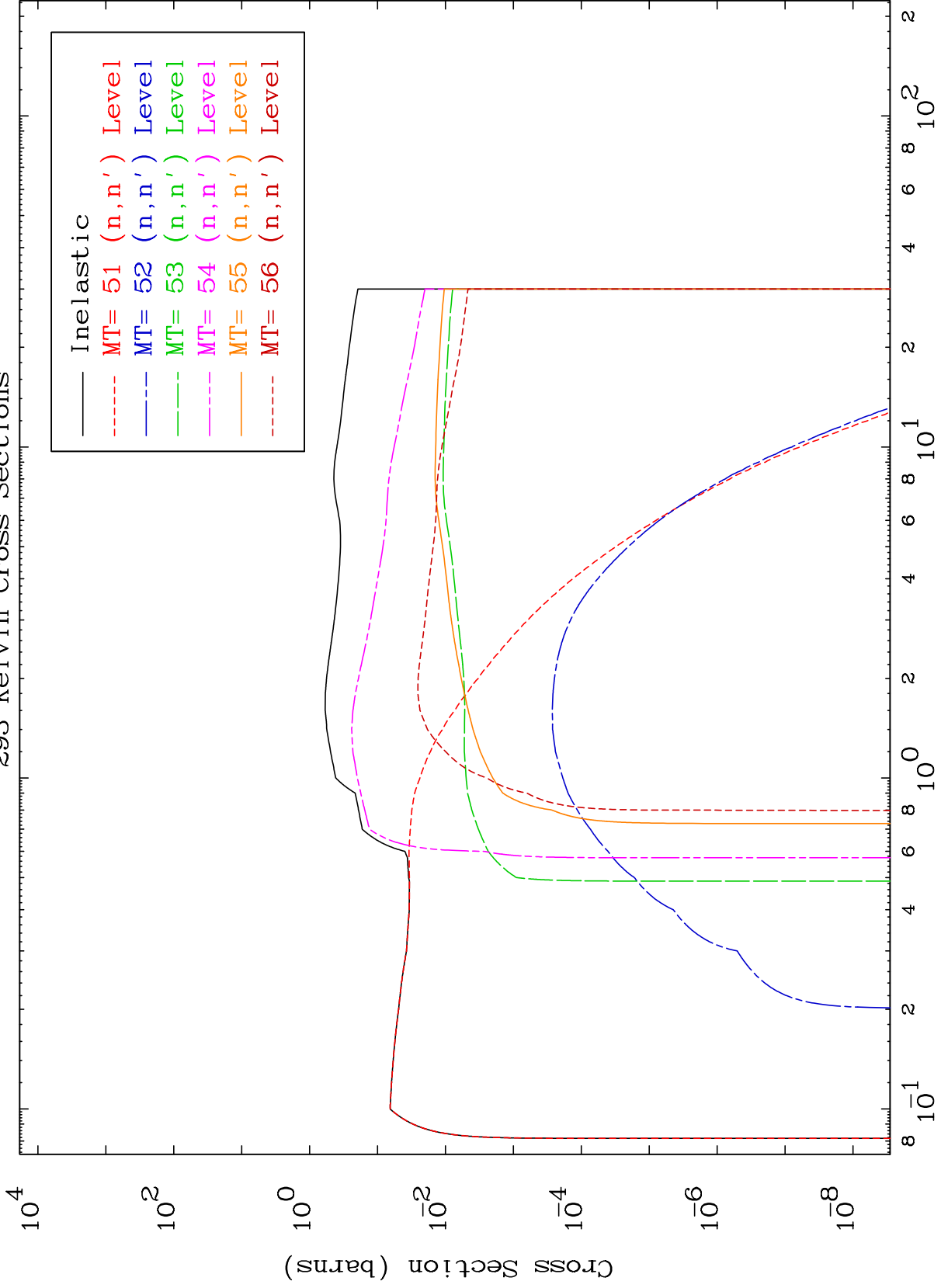


MAT 6780

(n,n') Levels

68-Er-147

293 Kelvin Cross Sections



Incident Energy (MeV)

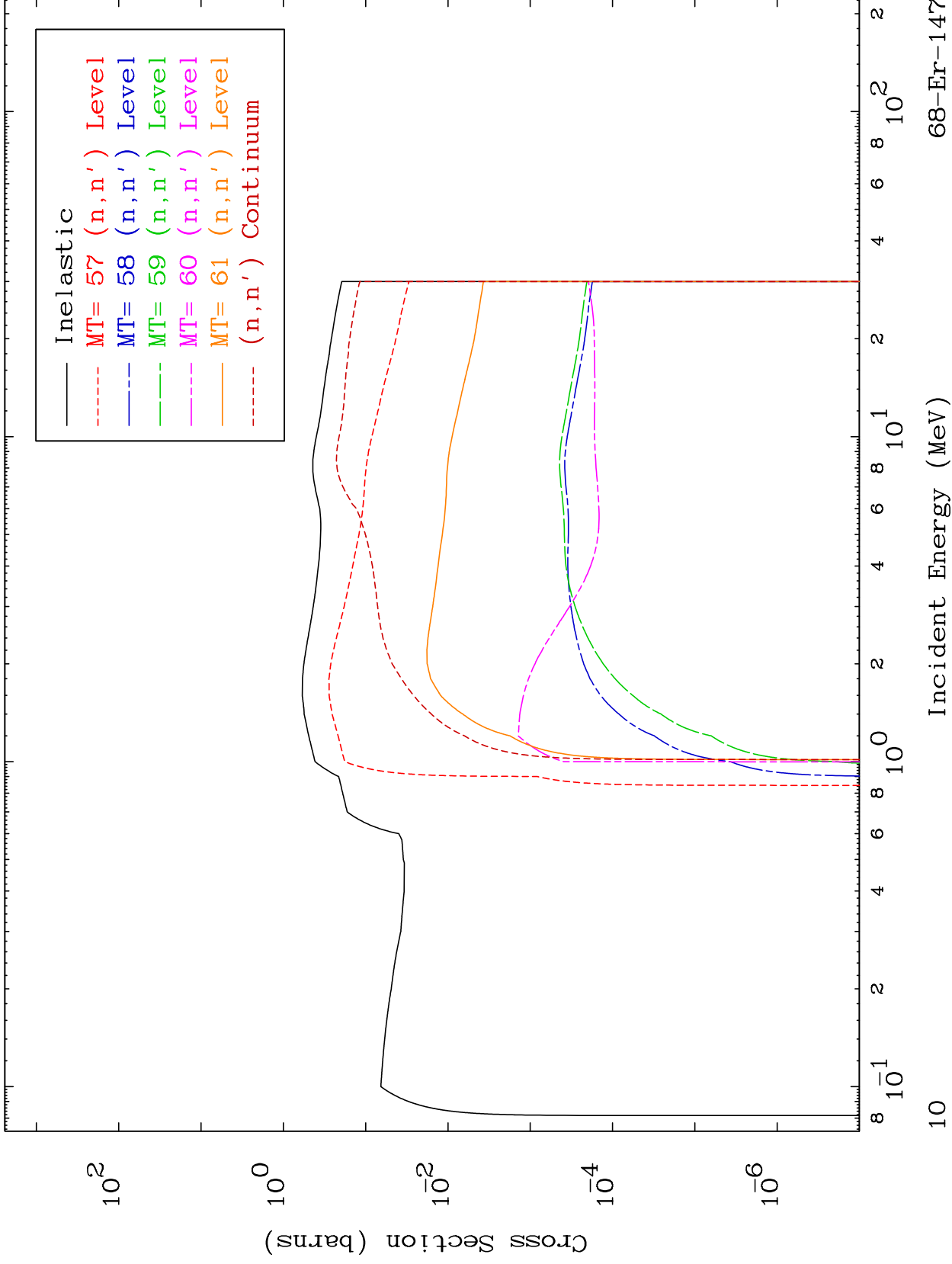
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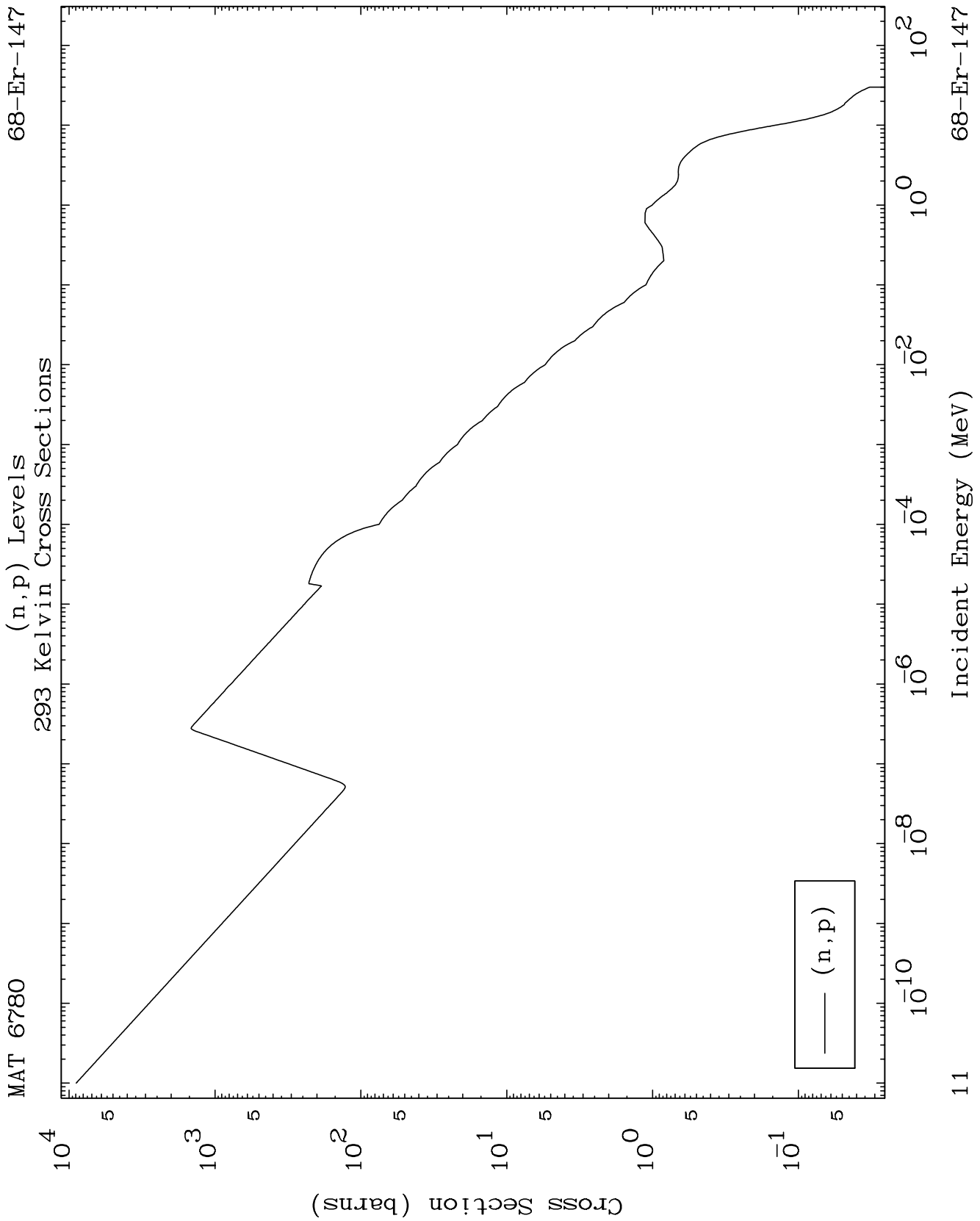
9

MAT 6780

(n,n') Levels
293 Kelvin Cross Sections

68-Er-147

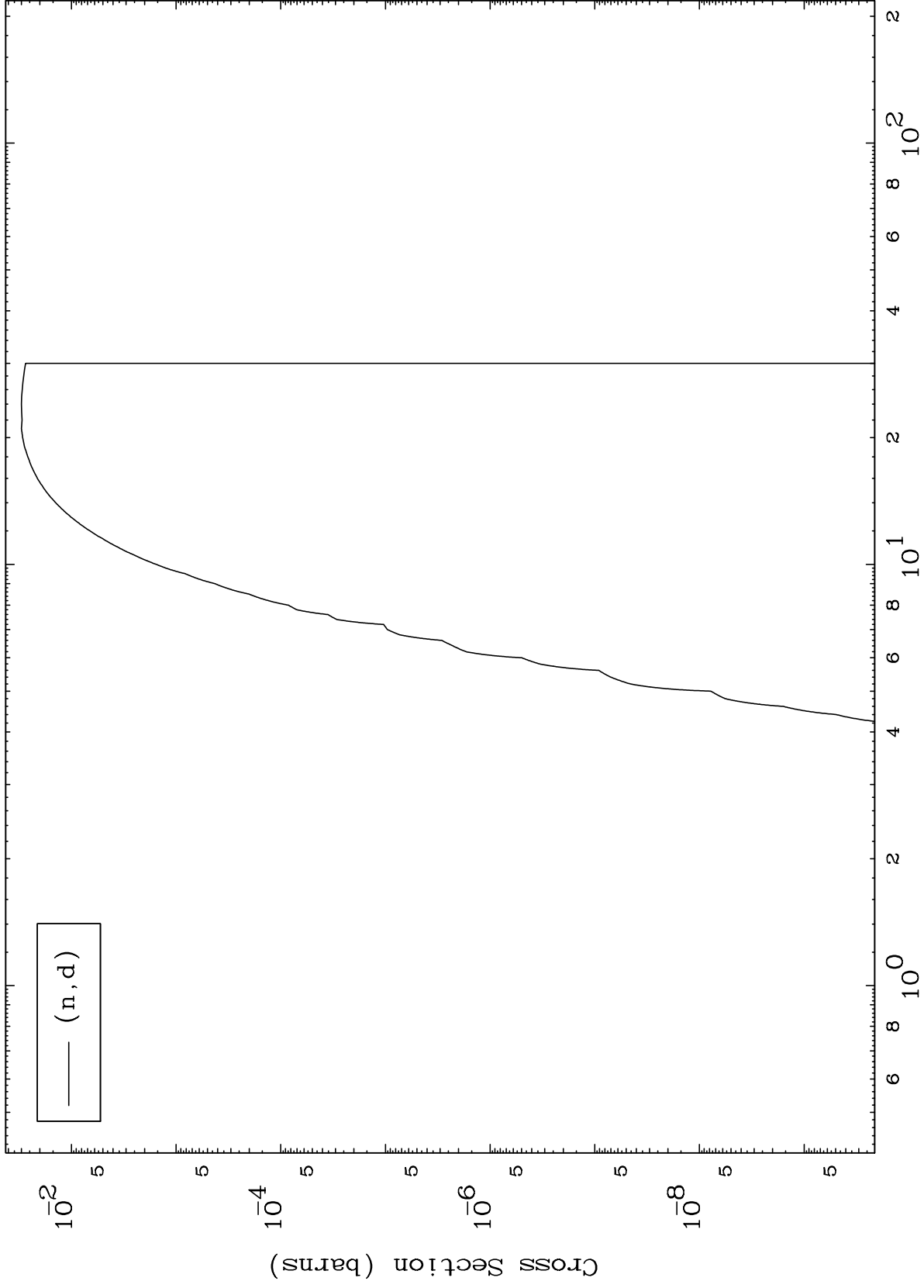




MAT 6780

(n,d) Levels
293 Kelvin Cross Sections

68-Er-147



12

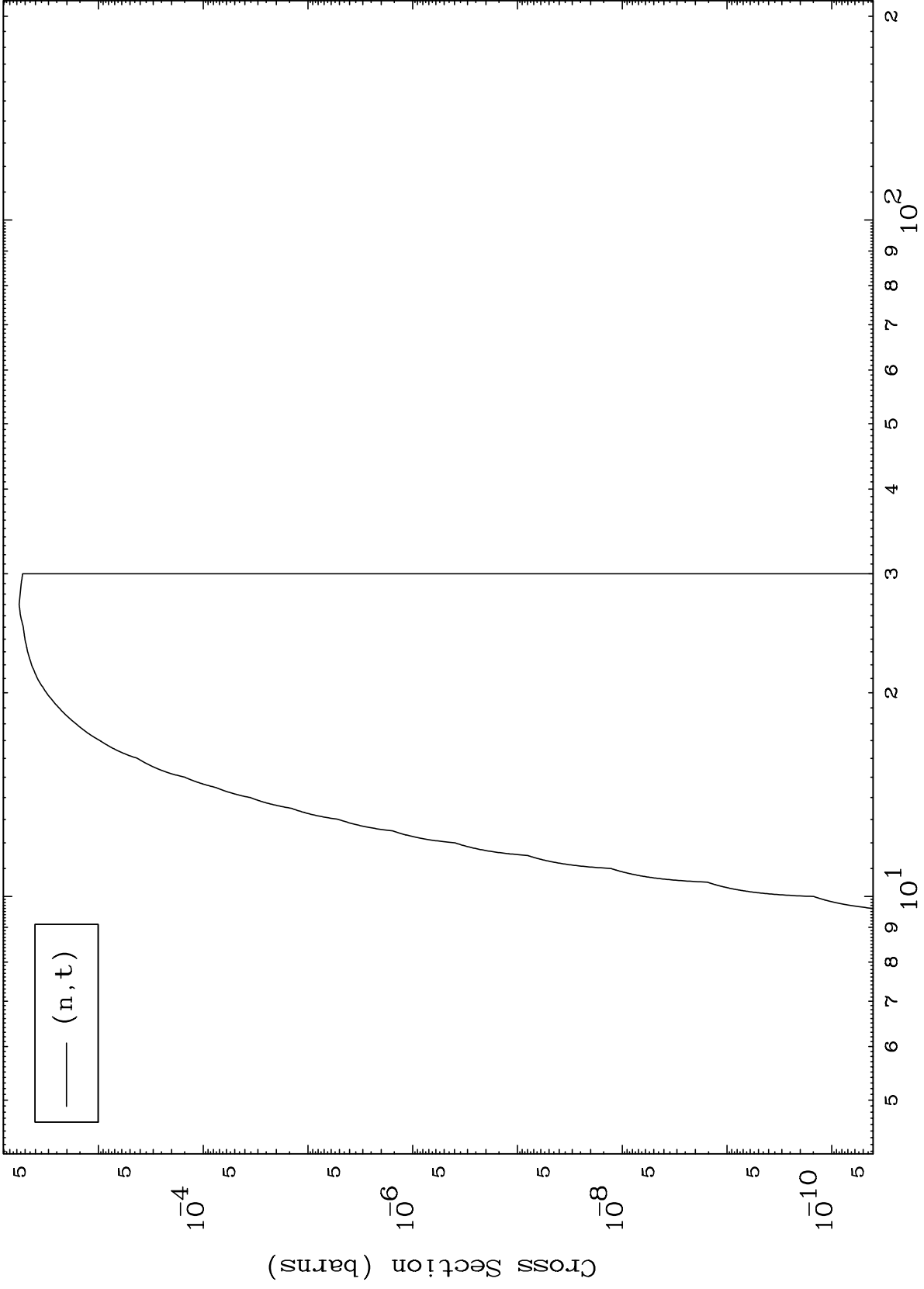
Incident Energy (MeV)

68-Er-147

MAT 6780

(n,t) Levels
293 Kelvin Cross Sections

68-Er-147



13

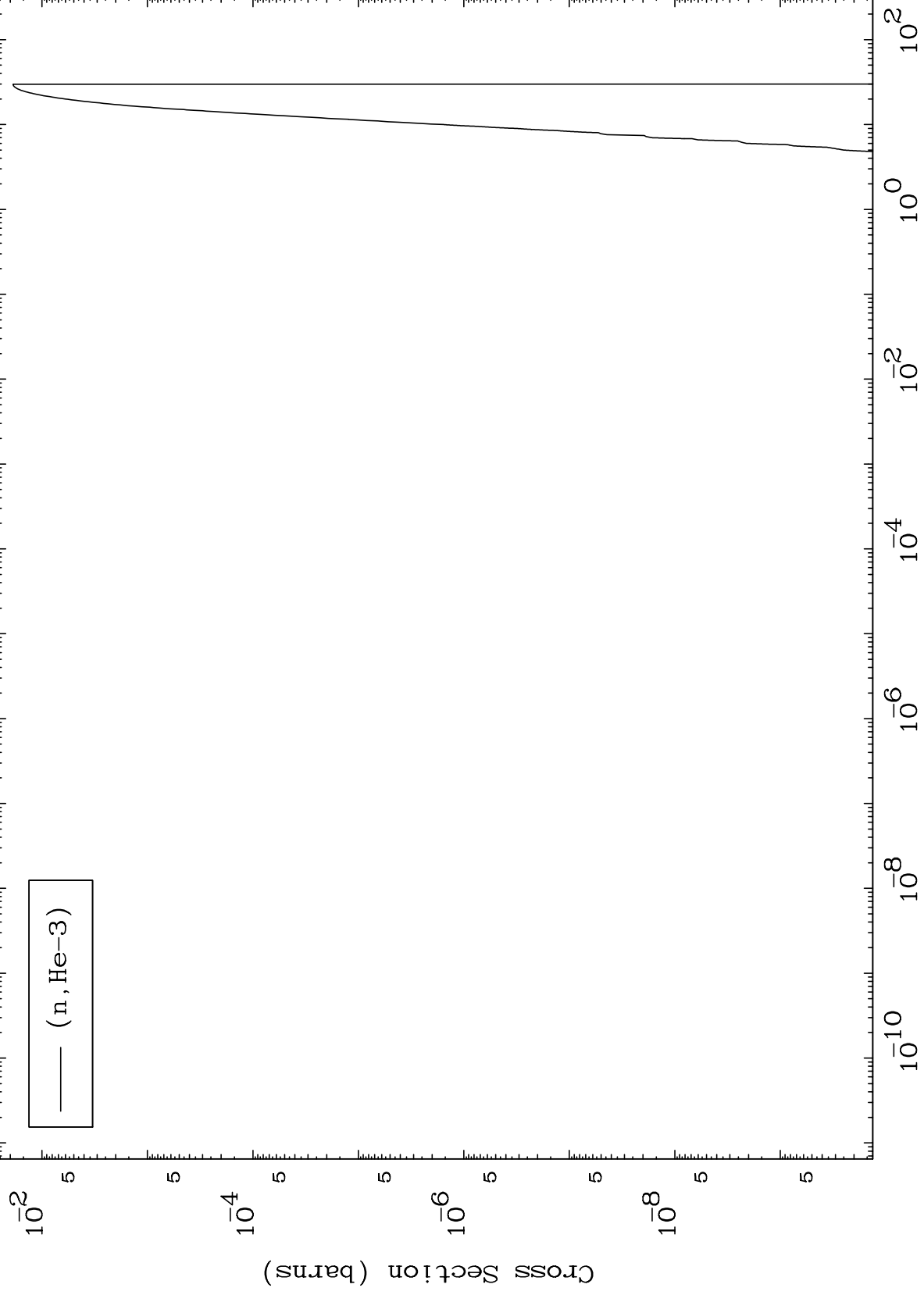
Incident Energy (MeV)

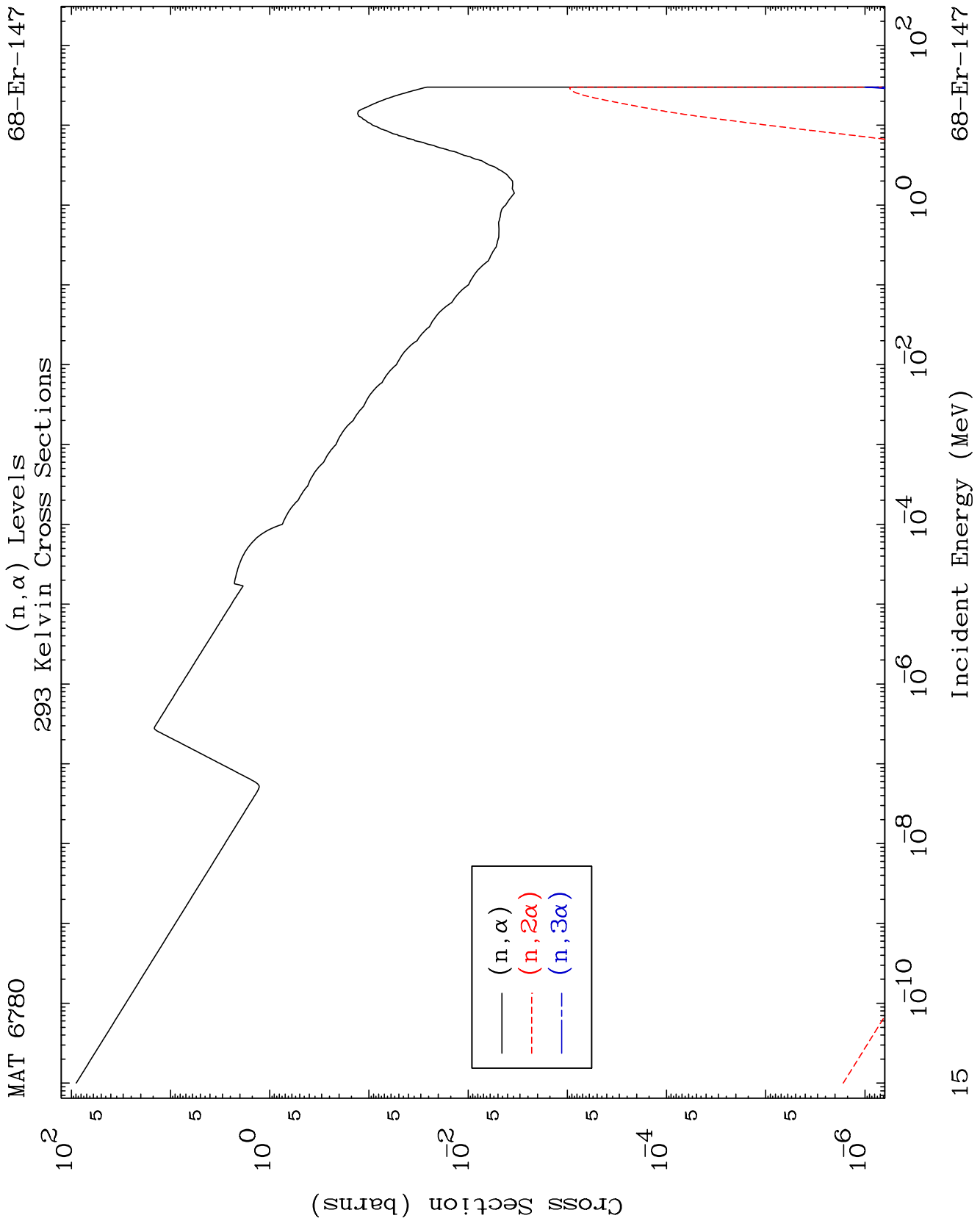
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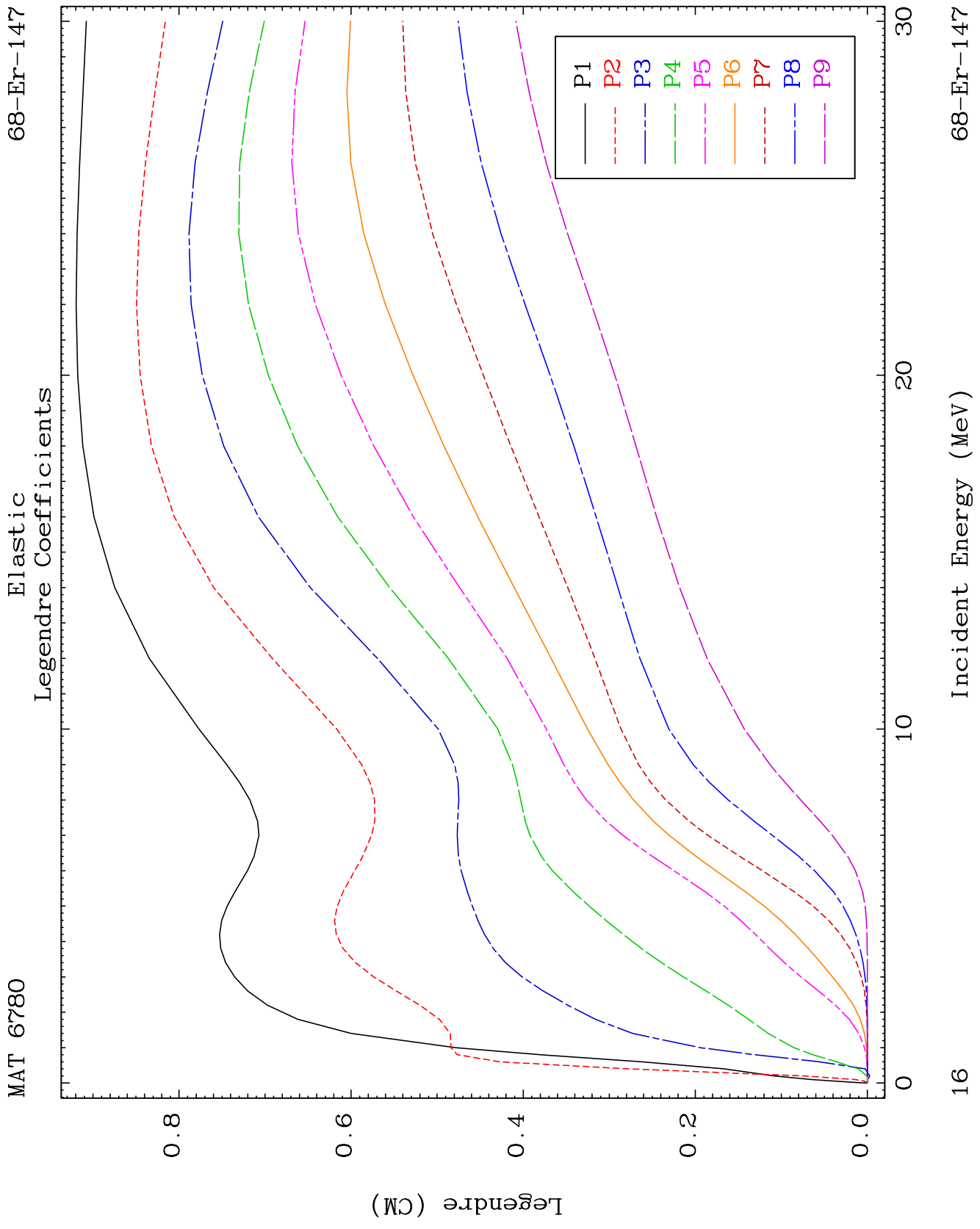
MAT 6780

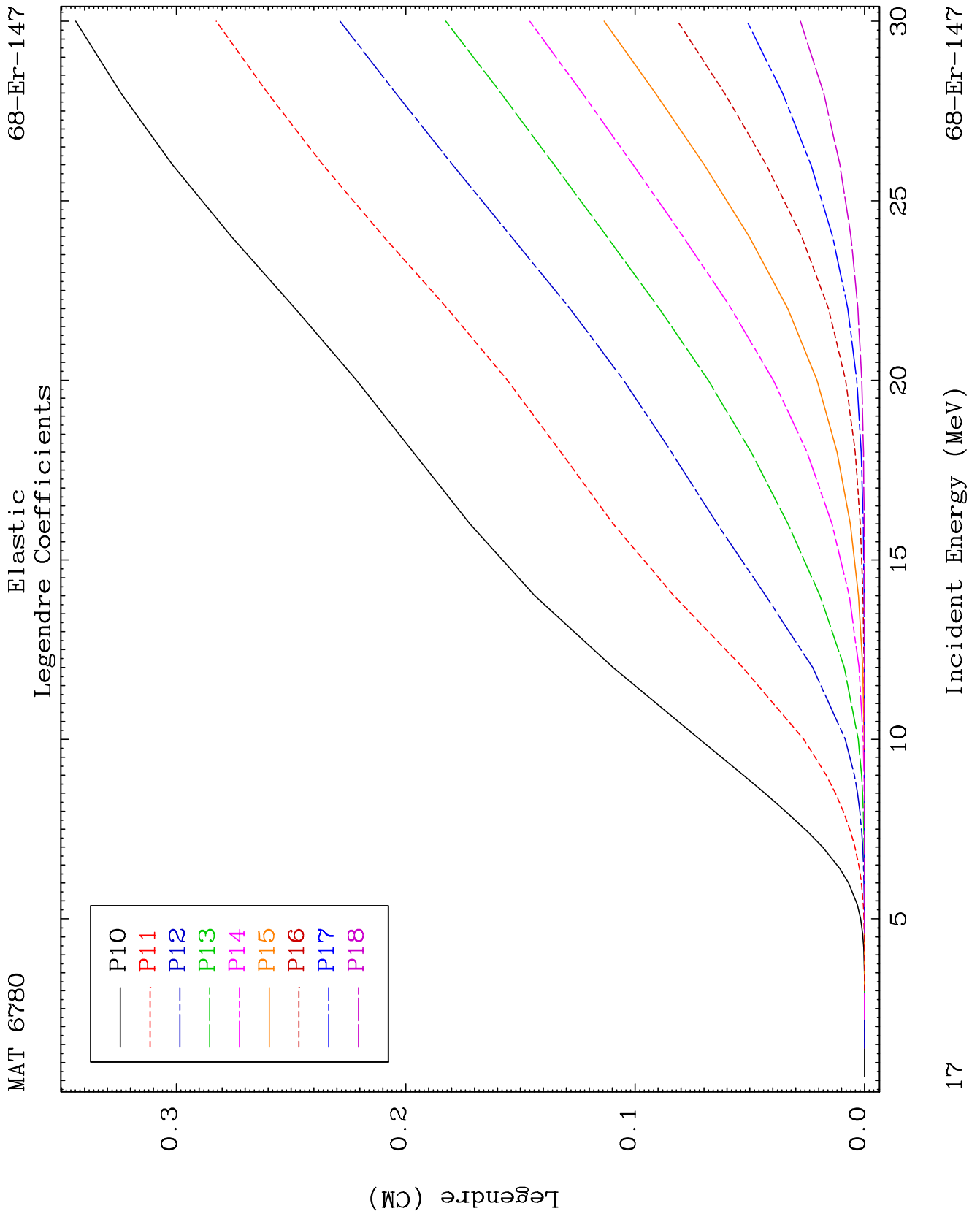
(n,He3) Levels
293 Kelvin Cross Sections

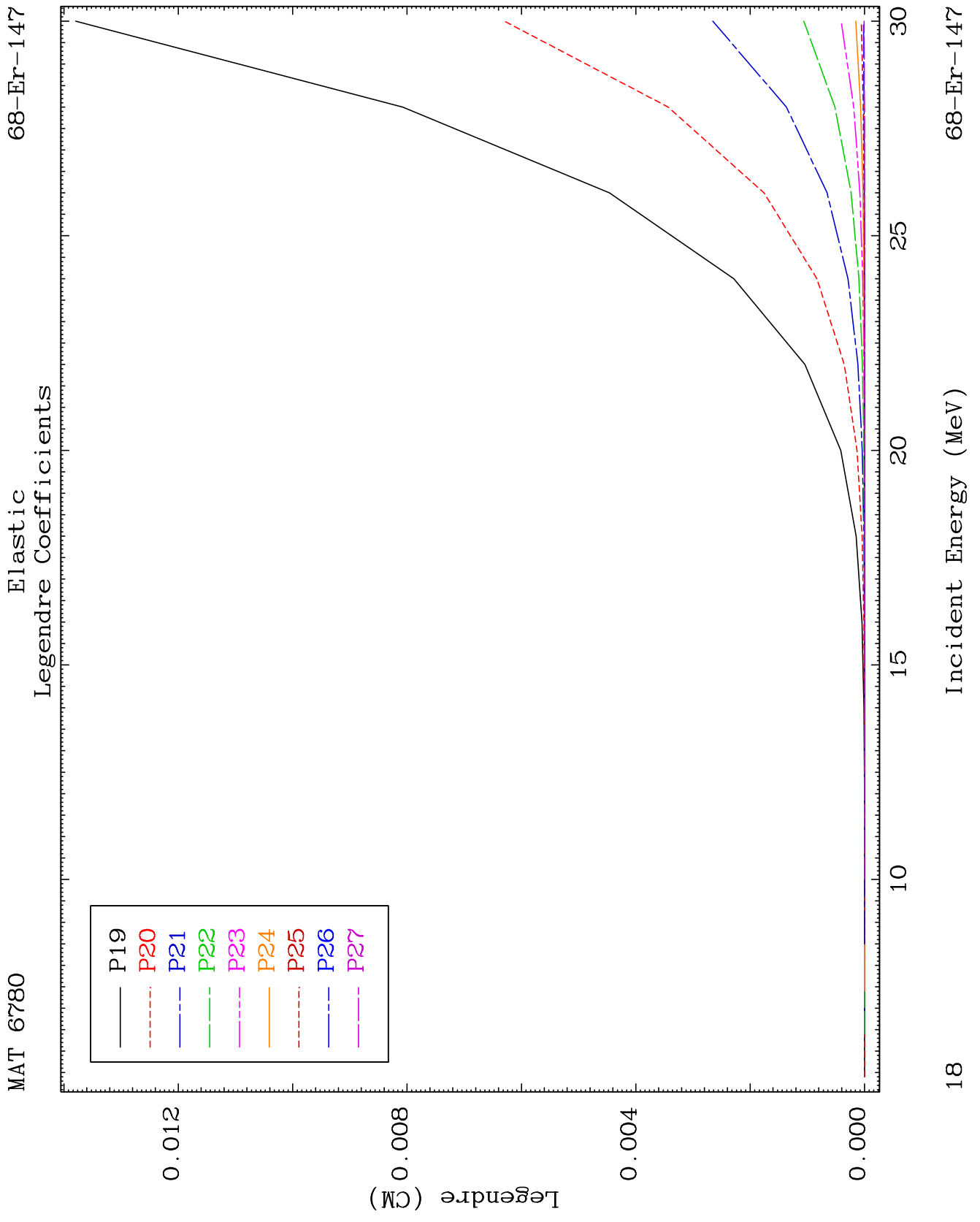
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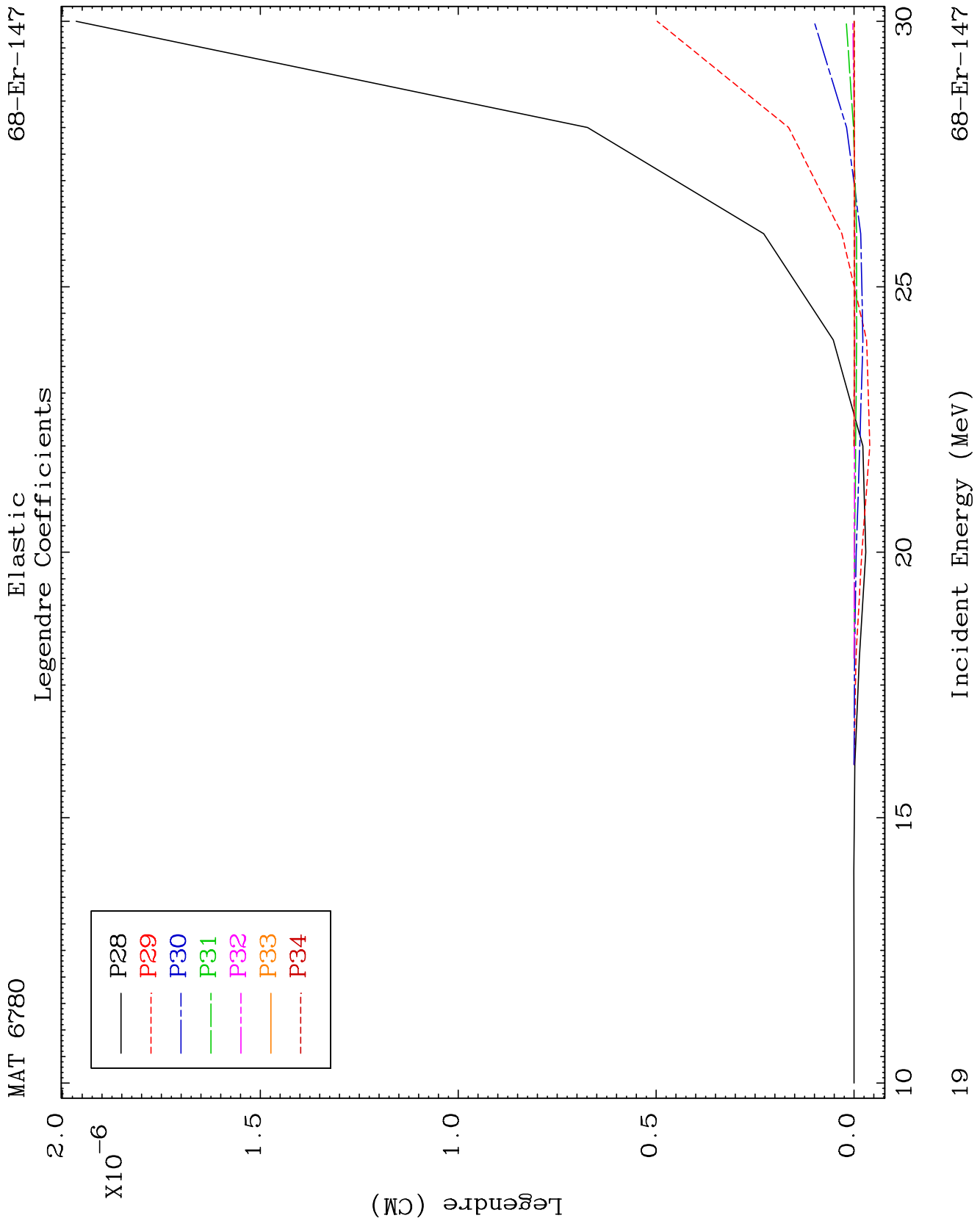


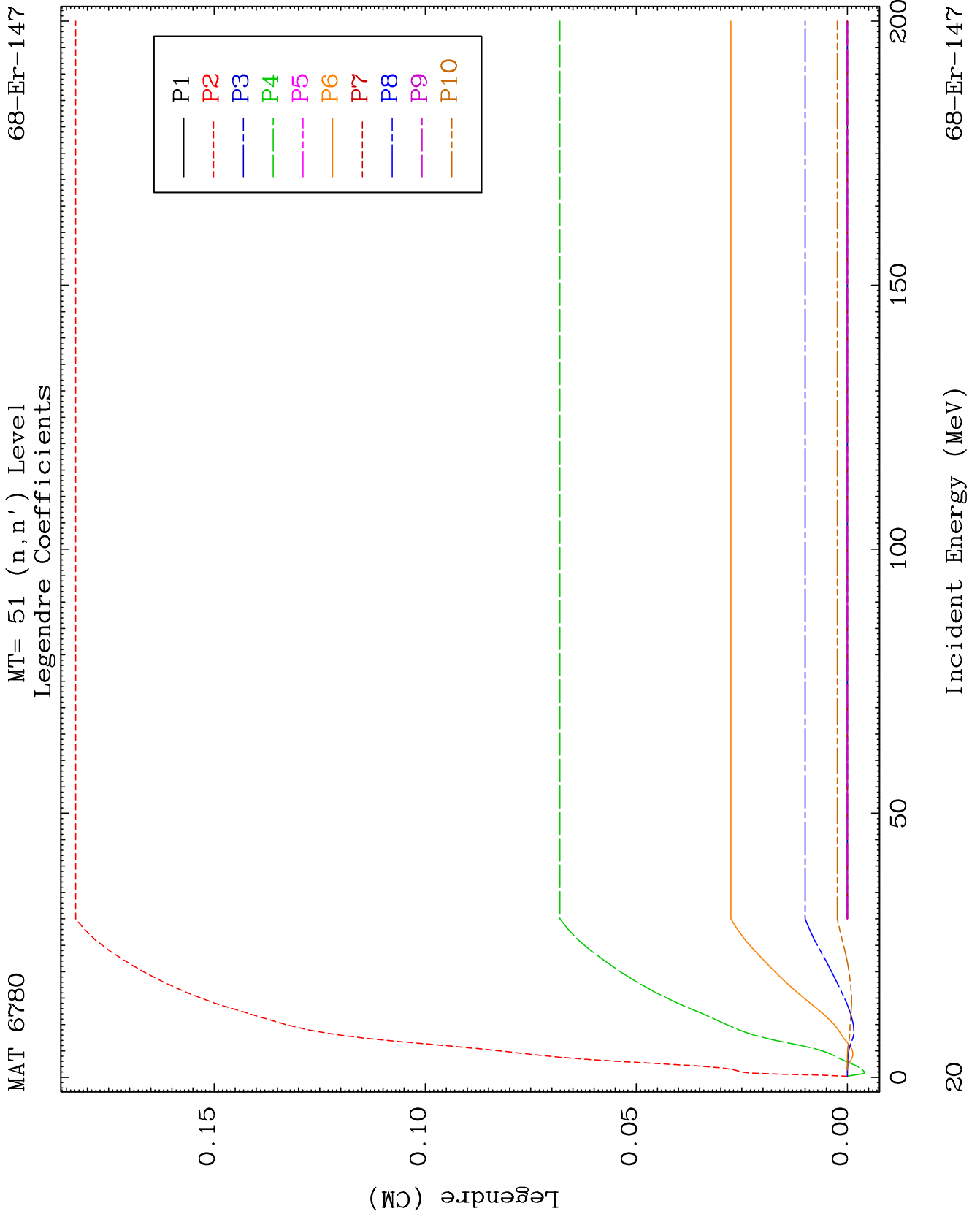








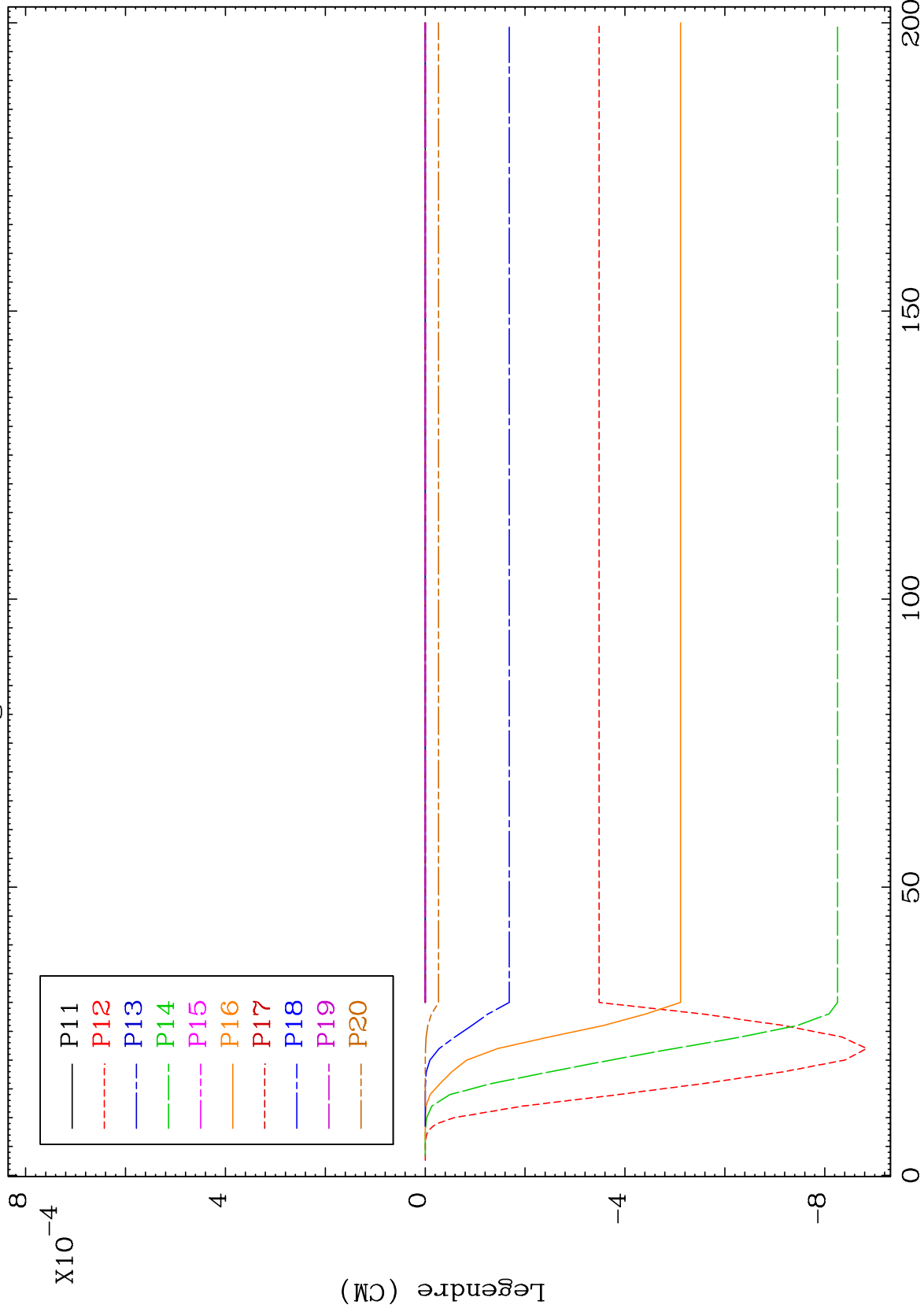




MAT 6780

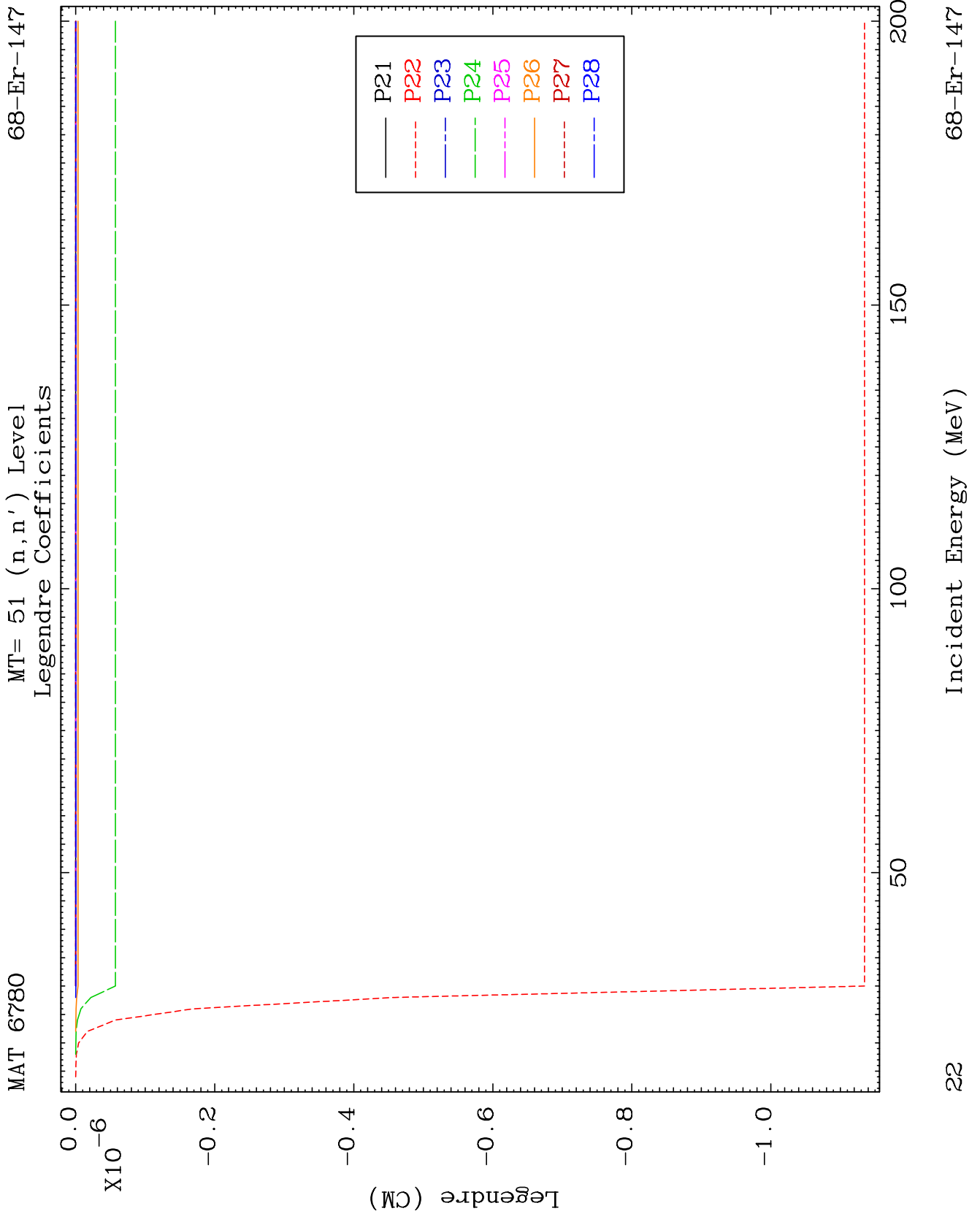
MT= 51 (n,n') Level
Legendre Coefficients

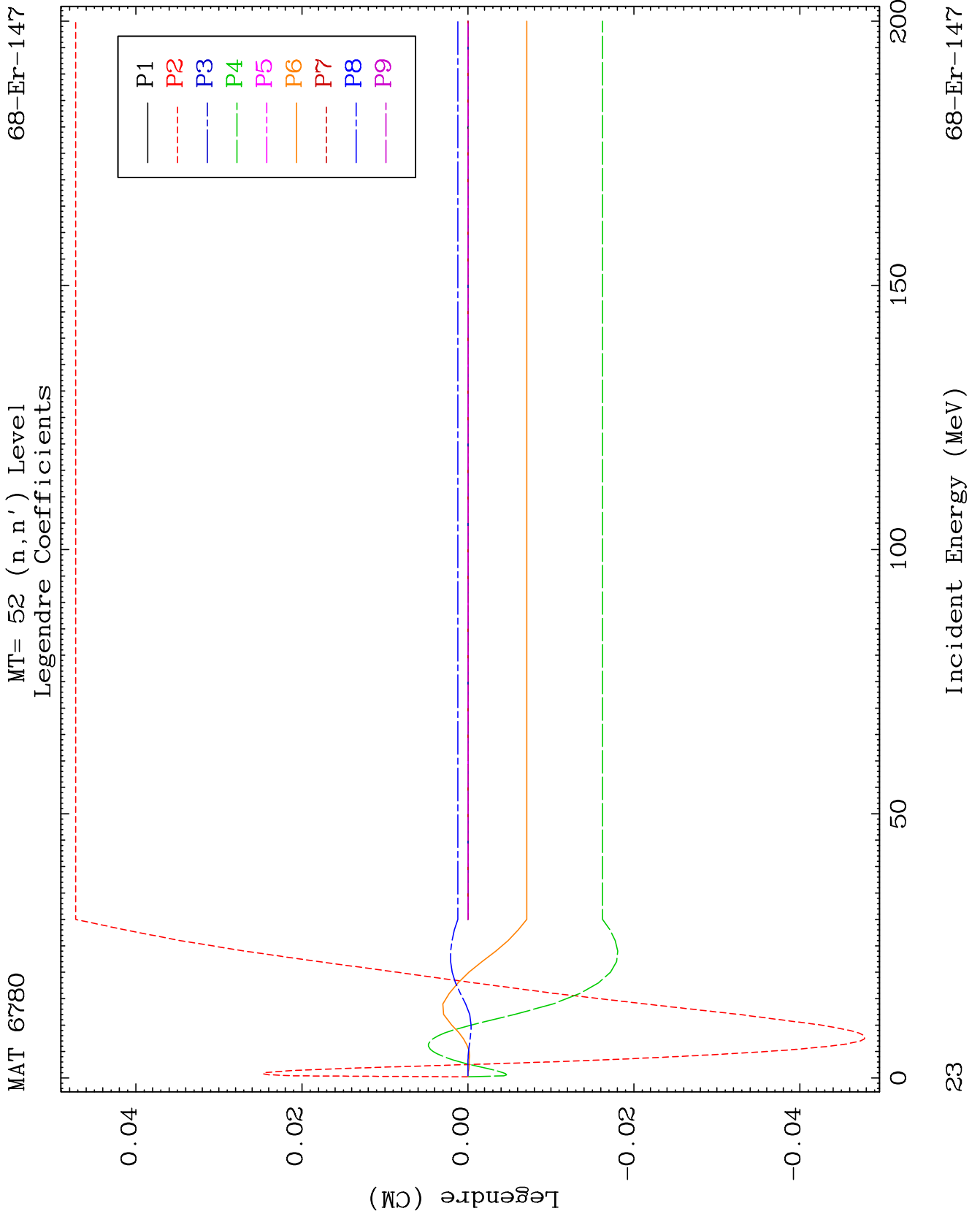
68-Er-147



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68-Er-147

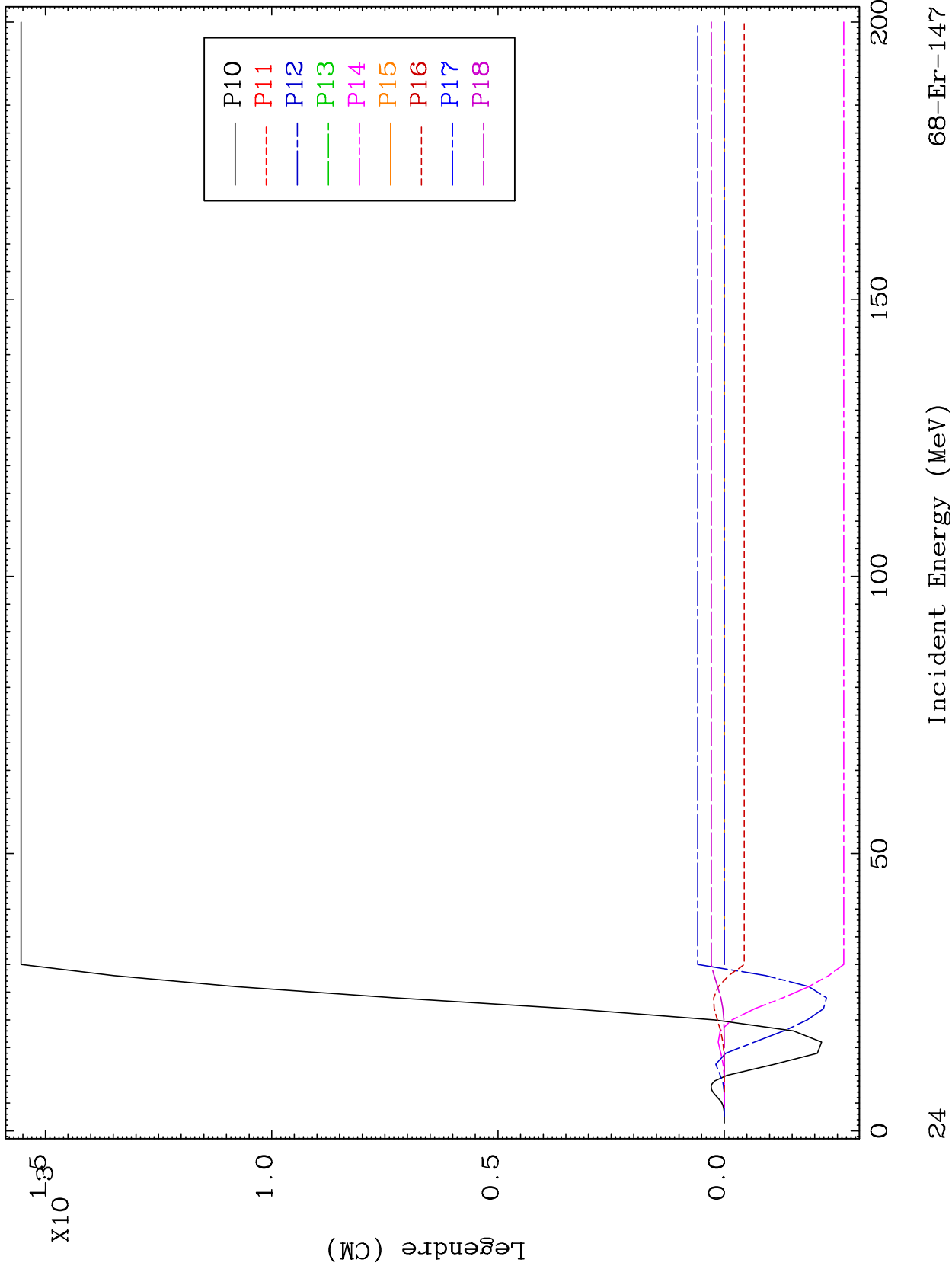




68-Er-147

Incident Energy (MeV)

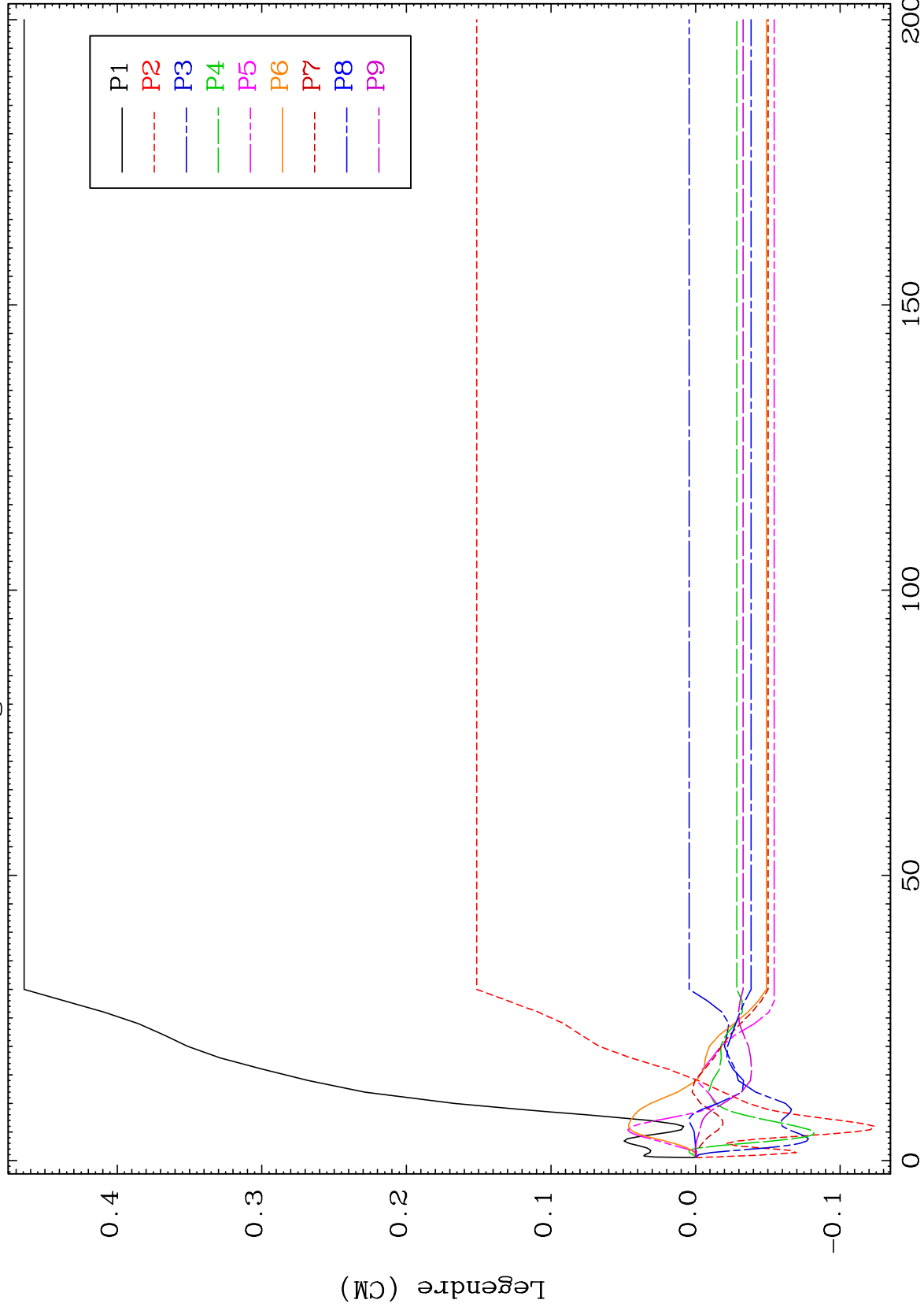
23



MAT 6780

MT= 53 (n,n') Level
Legendre Coefficients

68-Er-147



26

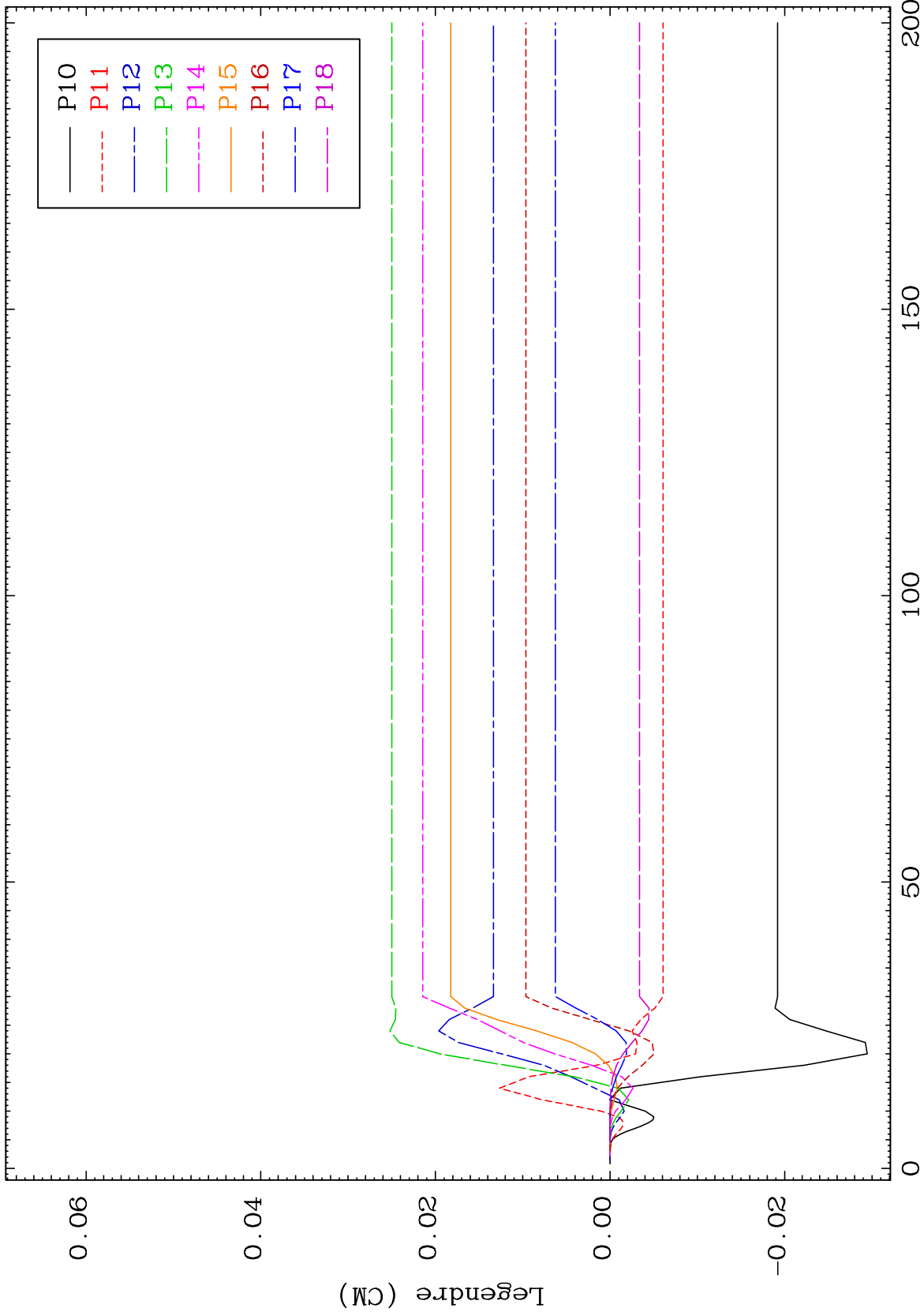
Incident Energy (MeV)

68-Er-147

MAT 6780

MT= 53 (n,n') Level
Legendre Coefficients

68-Er-147



27

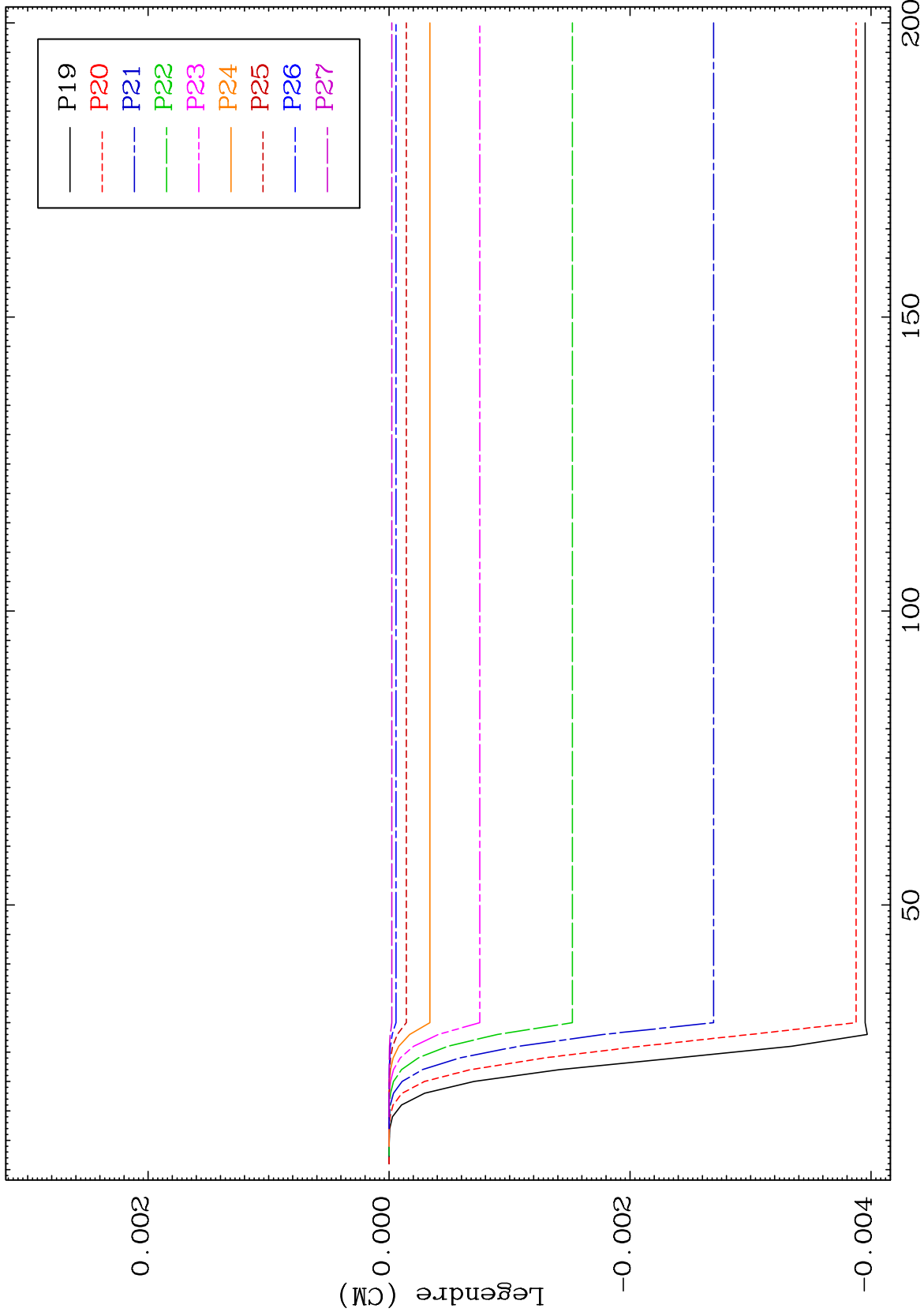
Incident Energy (MeV)

68-Er-147

MAT 6780

MT= 53 (n,n') Level
Legendre Coefficients

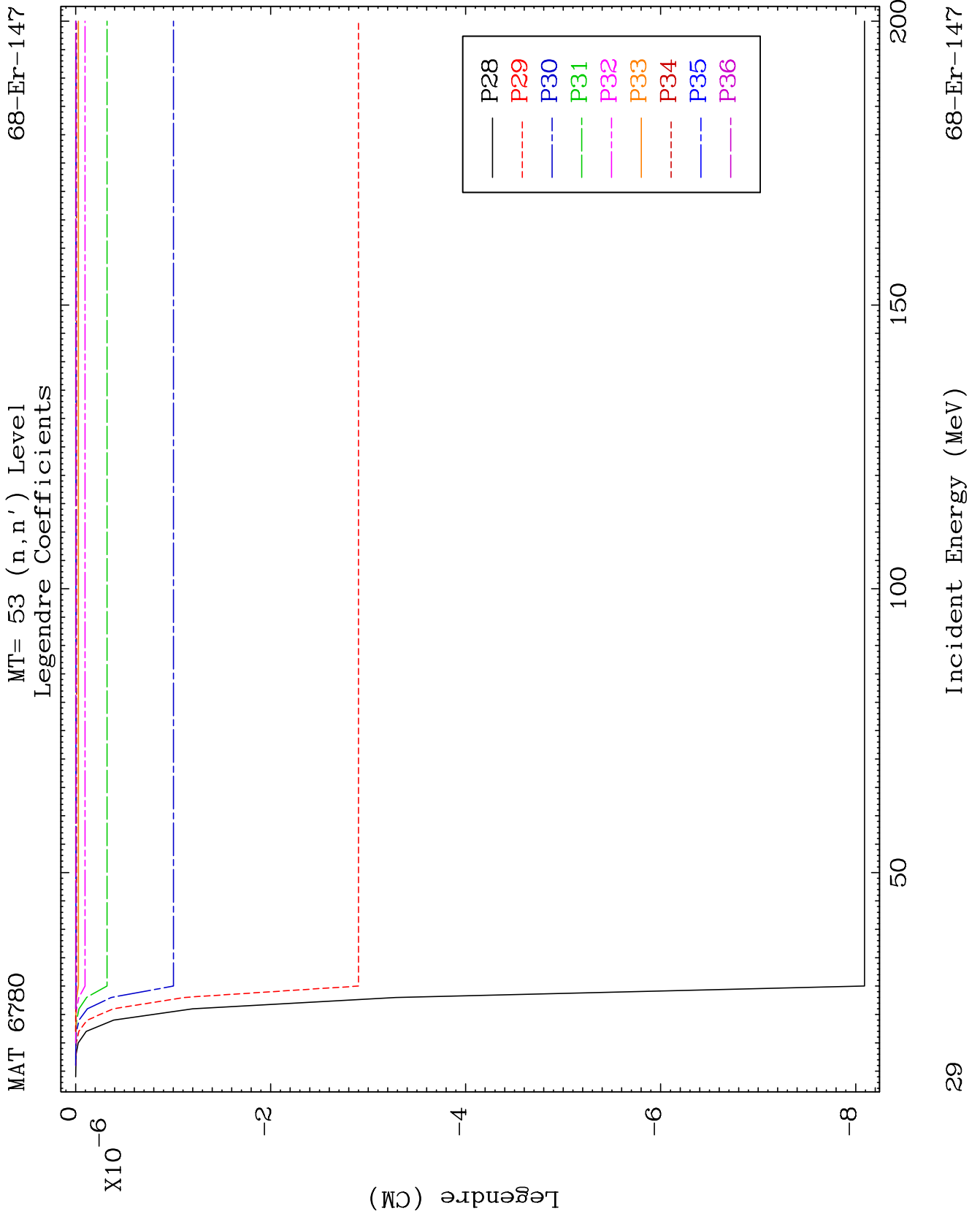
68-Er-147



28

Incident Energy (MeV)

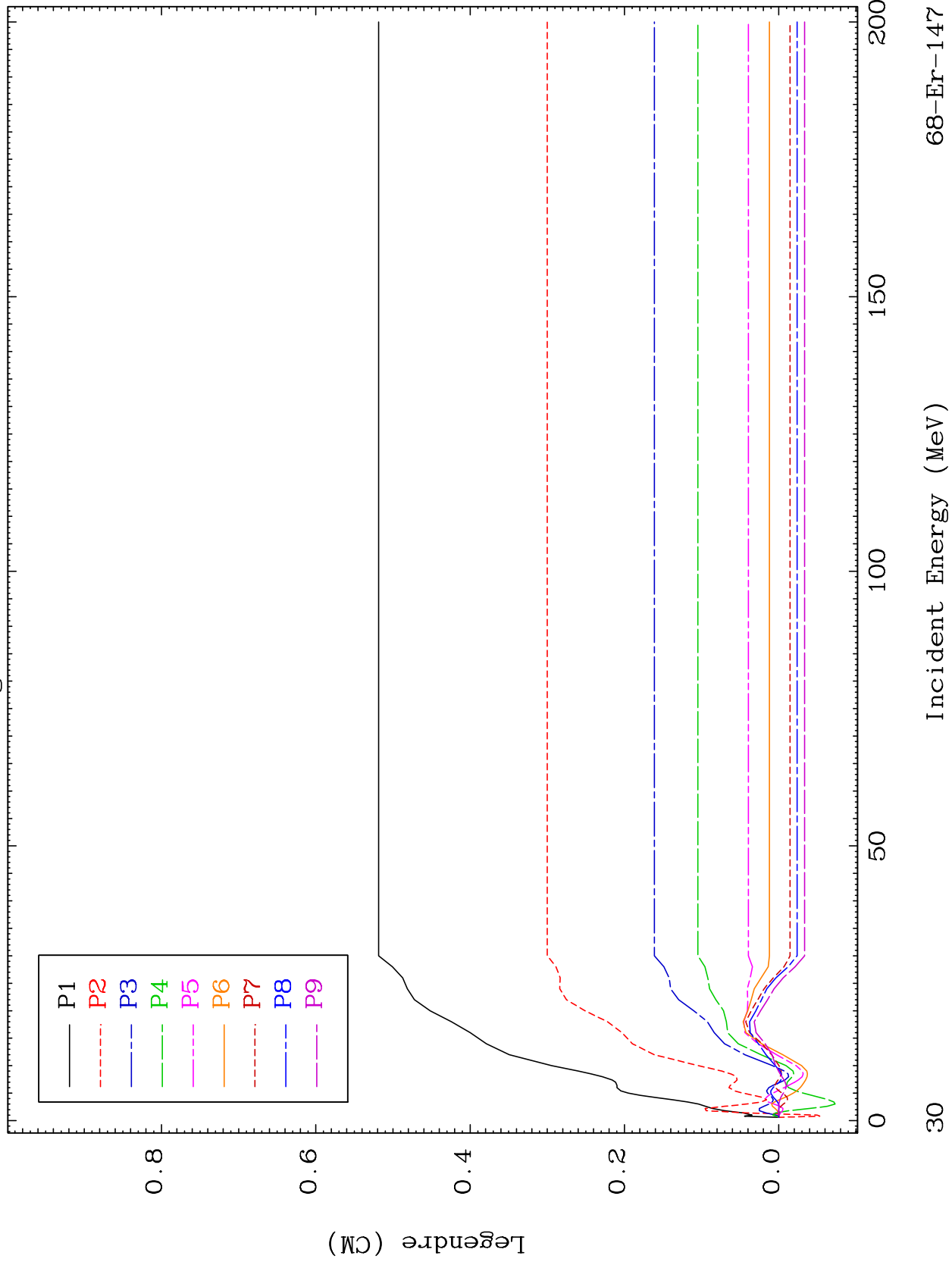
68-Er-147



MAT 6780

MT= 54 (n,n') Level
Legendre Coefficients

68-Er-147



68-Er-147

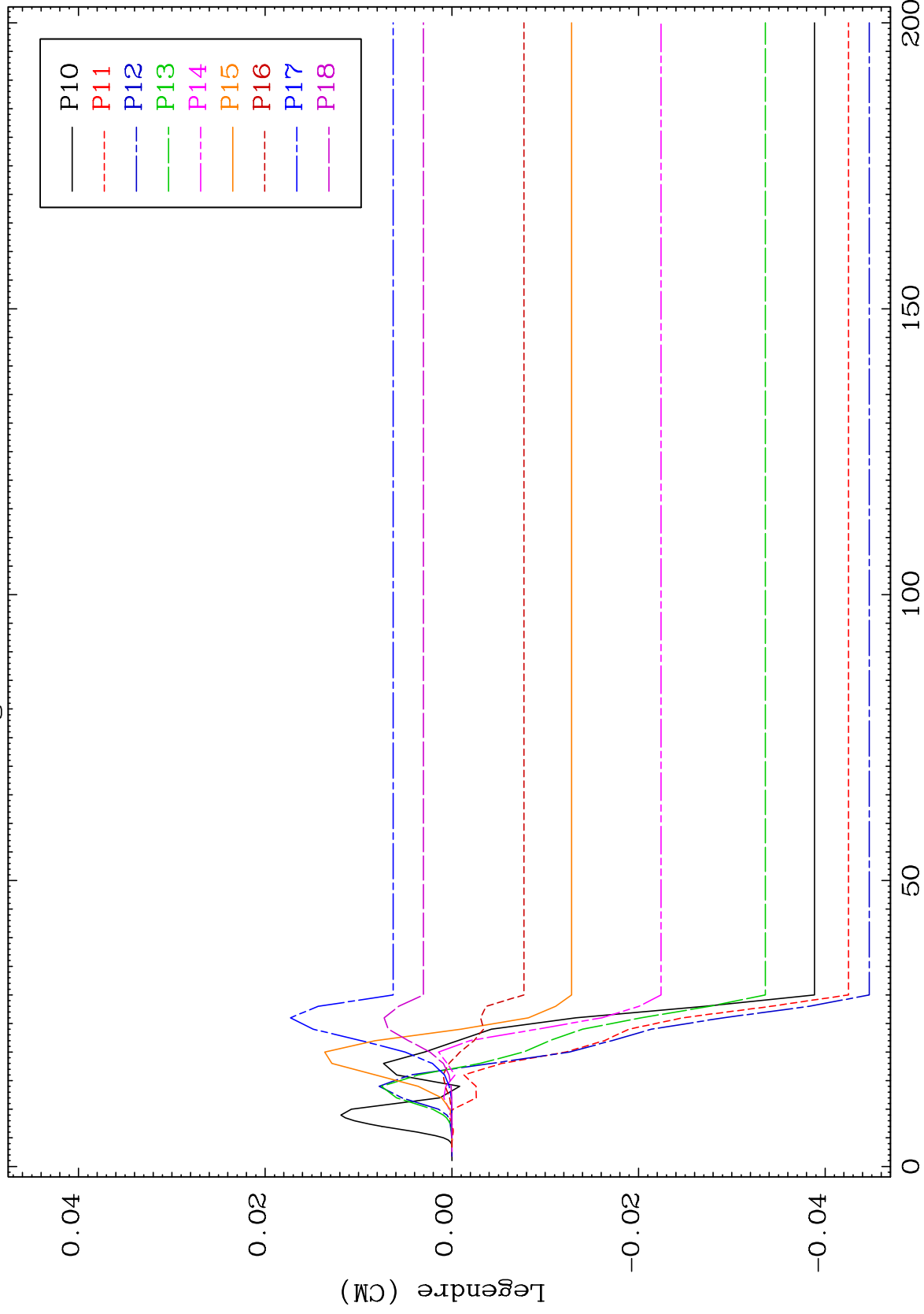
Incident Energy (MeV)

30

MAT 6780

MT= 54 (n,n') Level
Legendre Coefficients

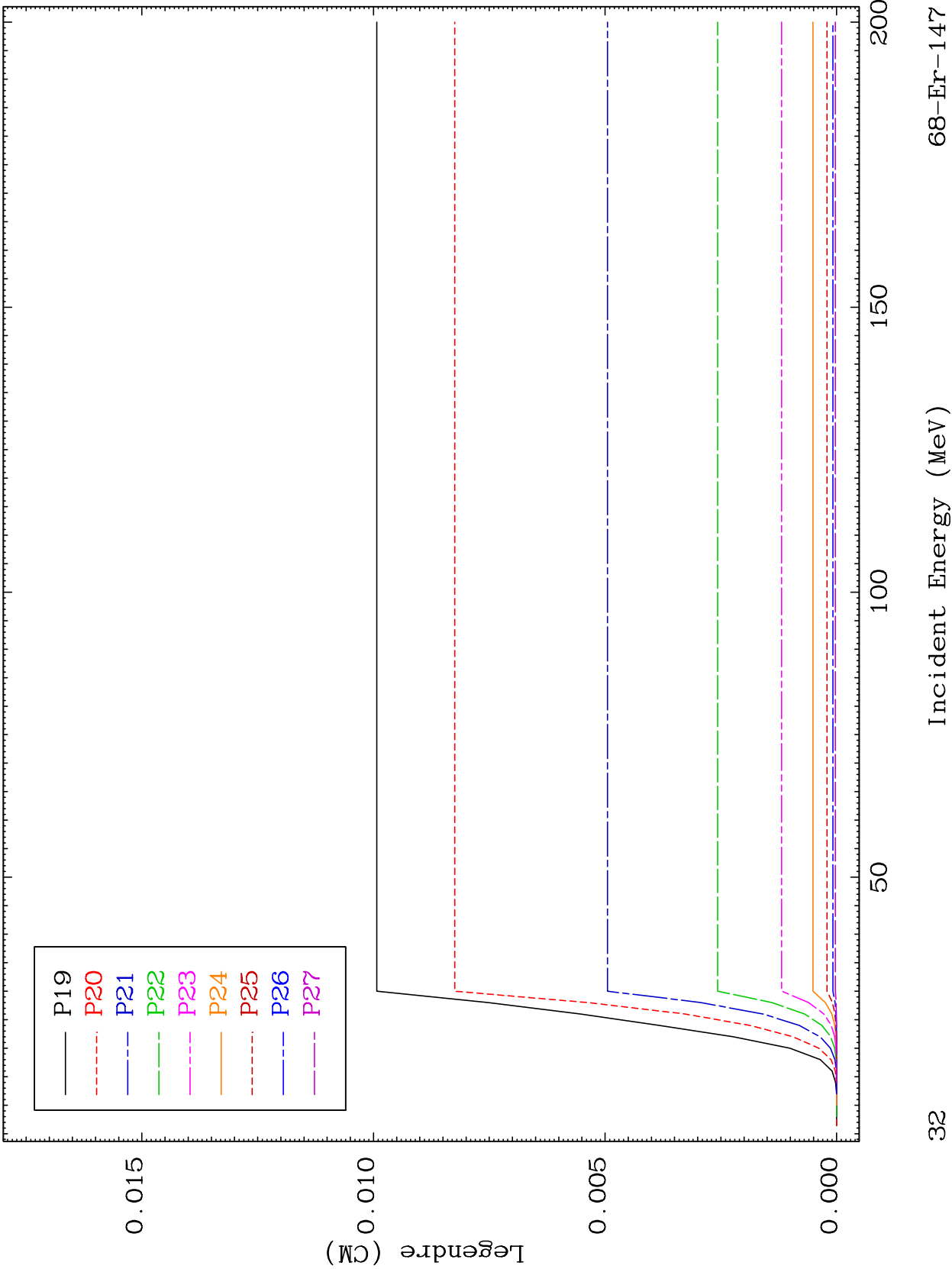
68-Er-147



31

Incident Energy (MeV)

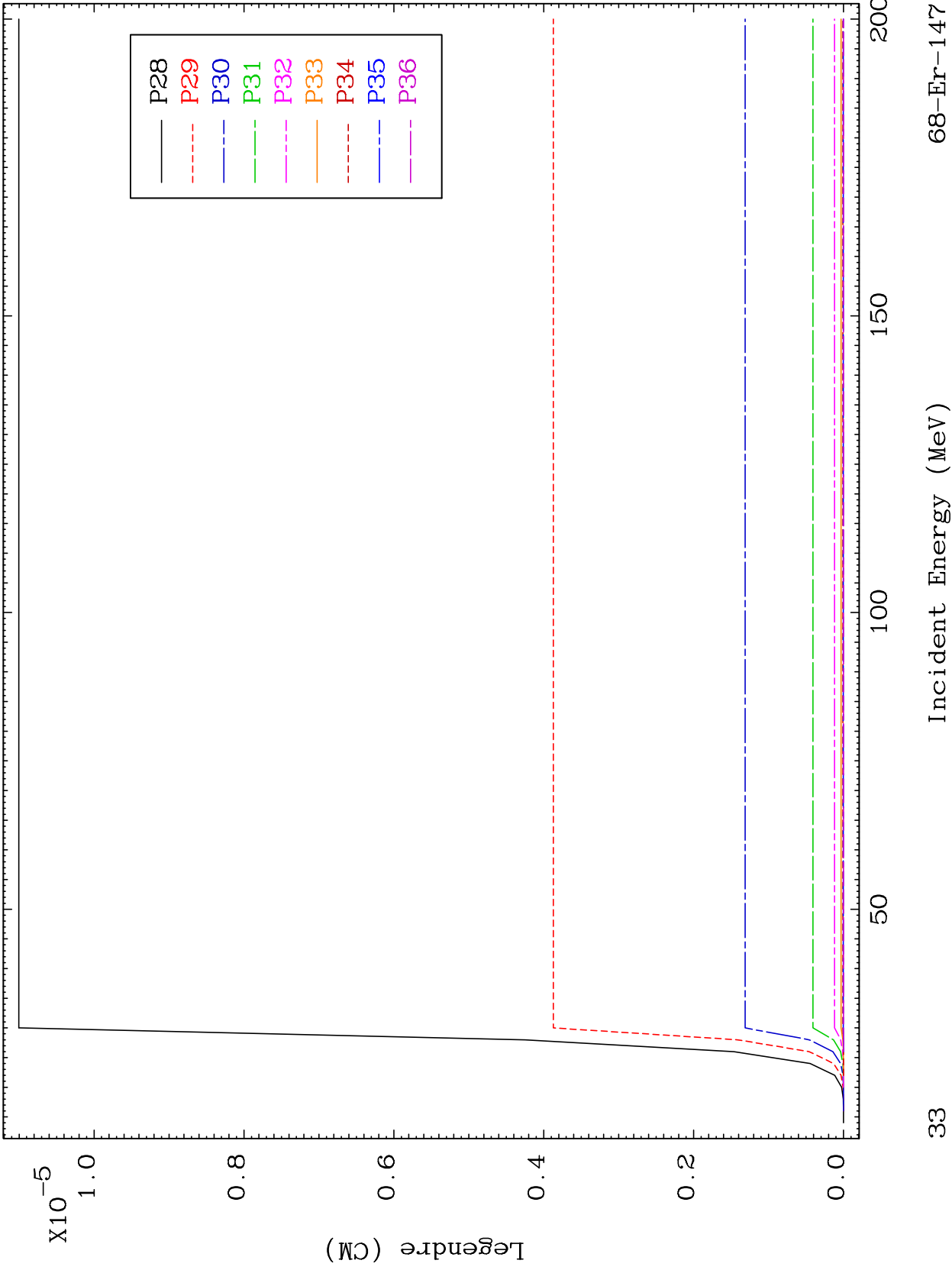
68-Er-147



MAT 6780

MT= 54 (n,n') Level
Legendre Coefficients

68-Er-147



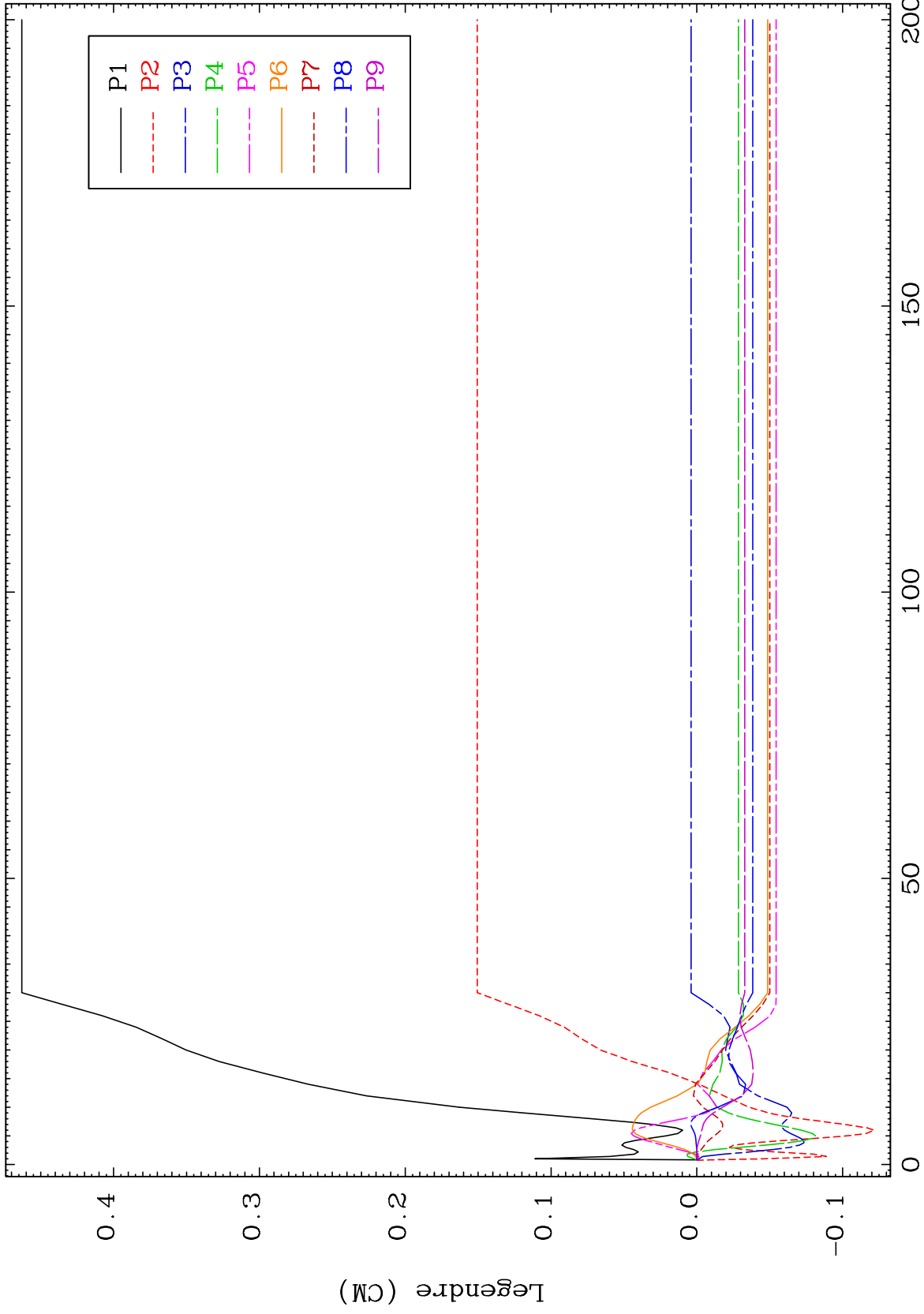
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68-Er-147

MAT 6780

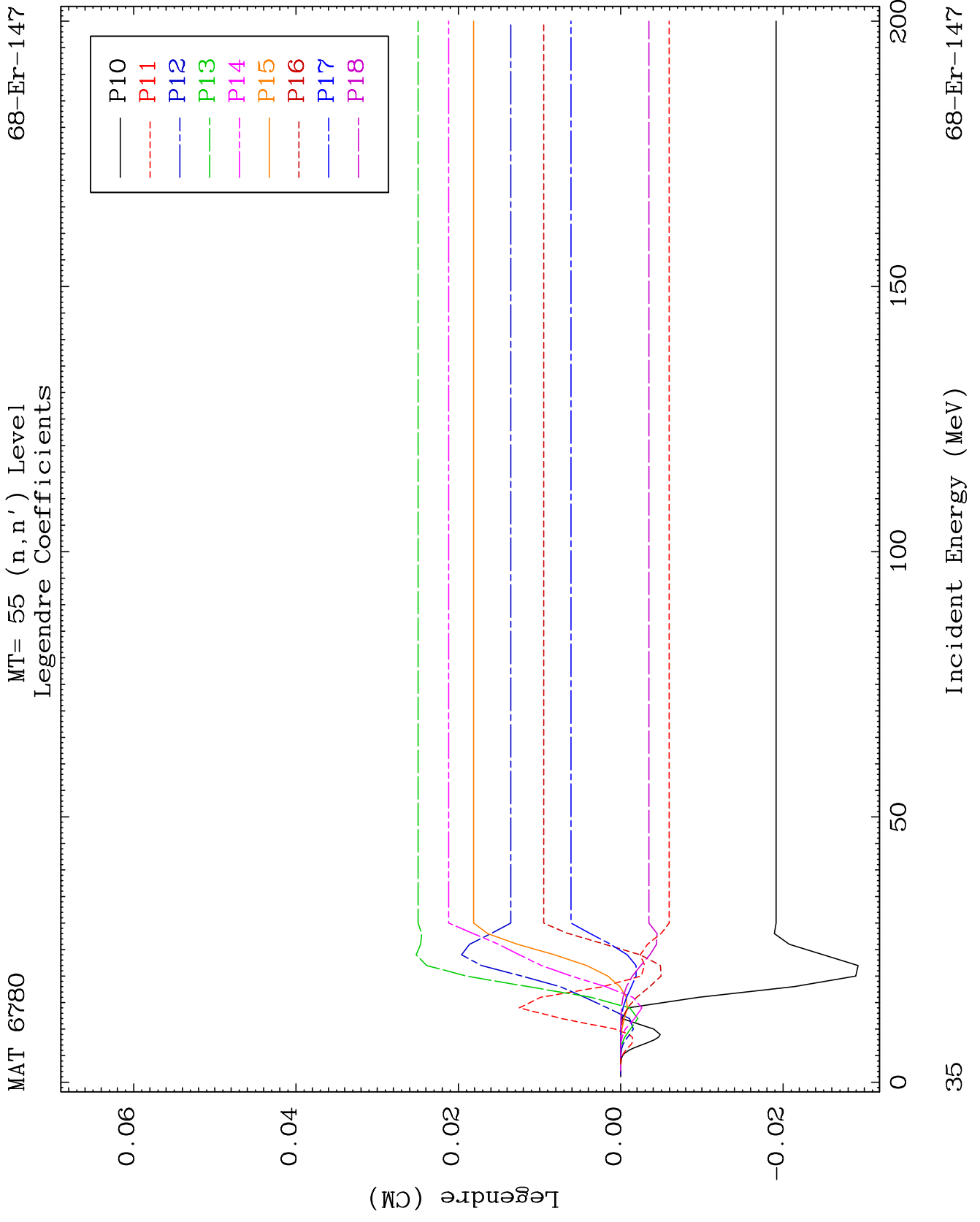
MT= 55 (n,n') Level
Legendre Coefficients

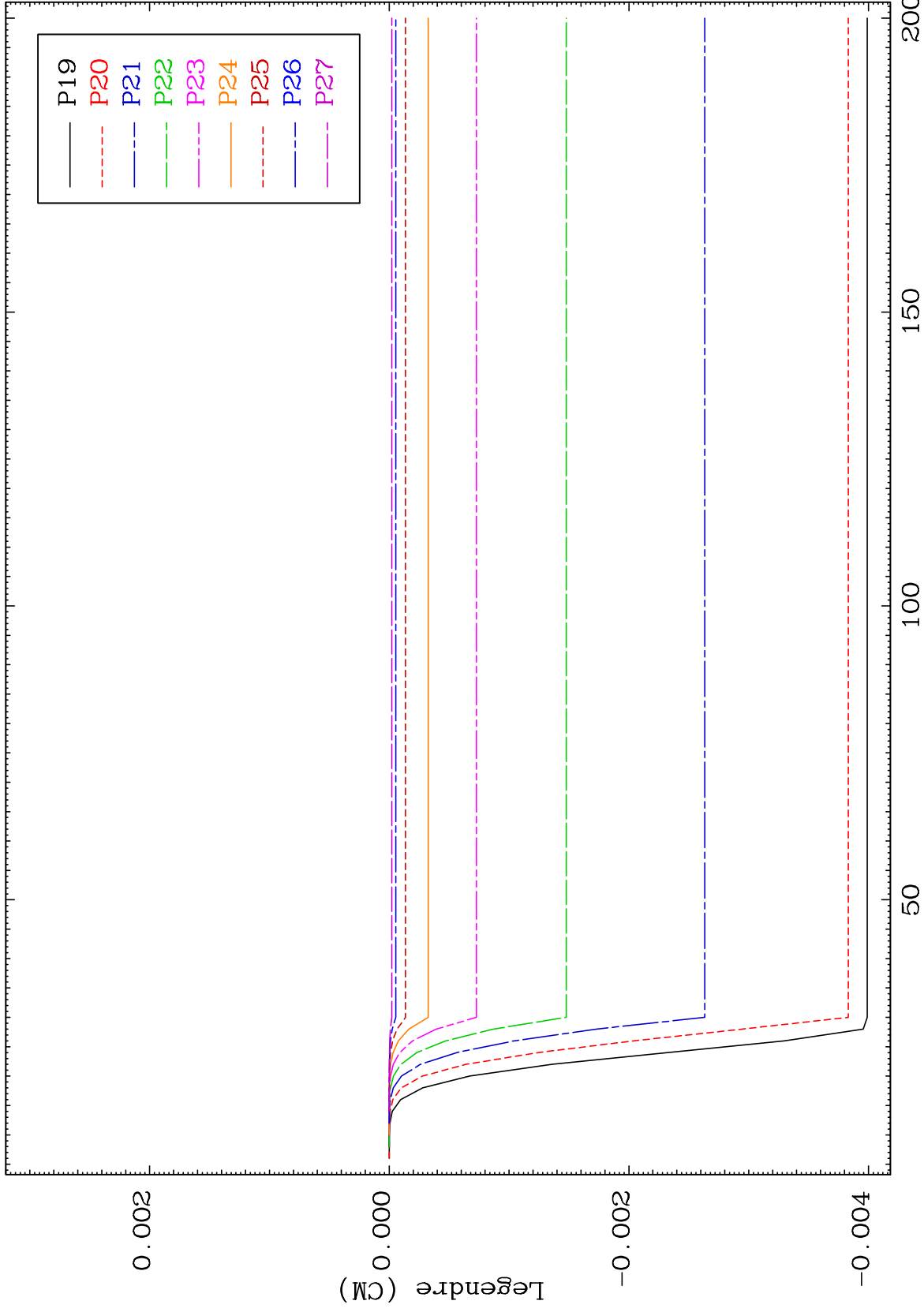
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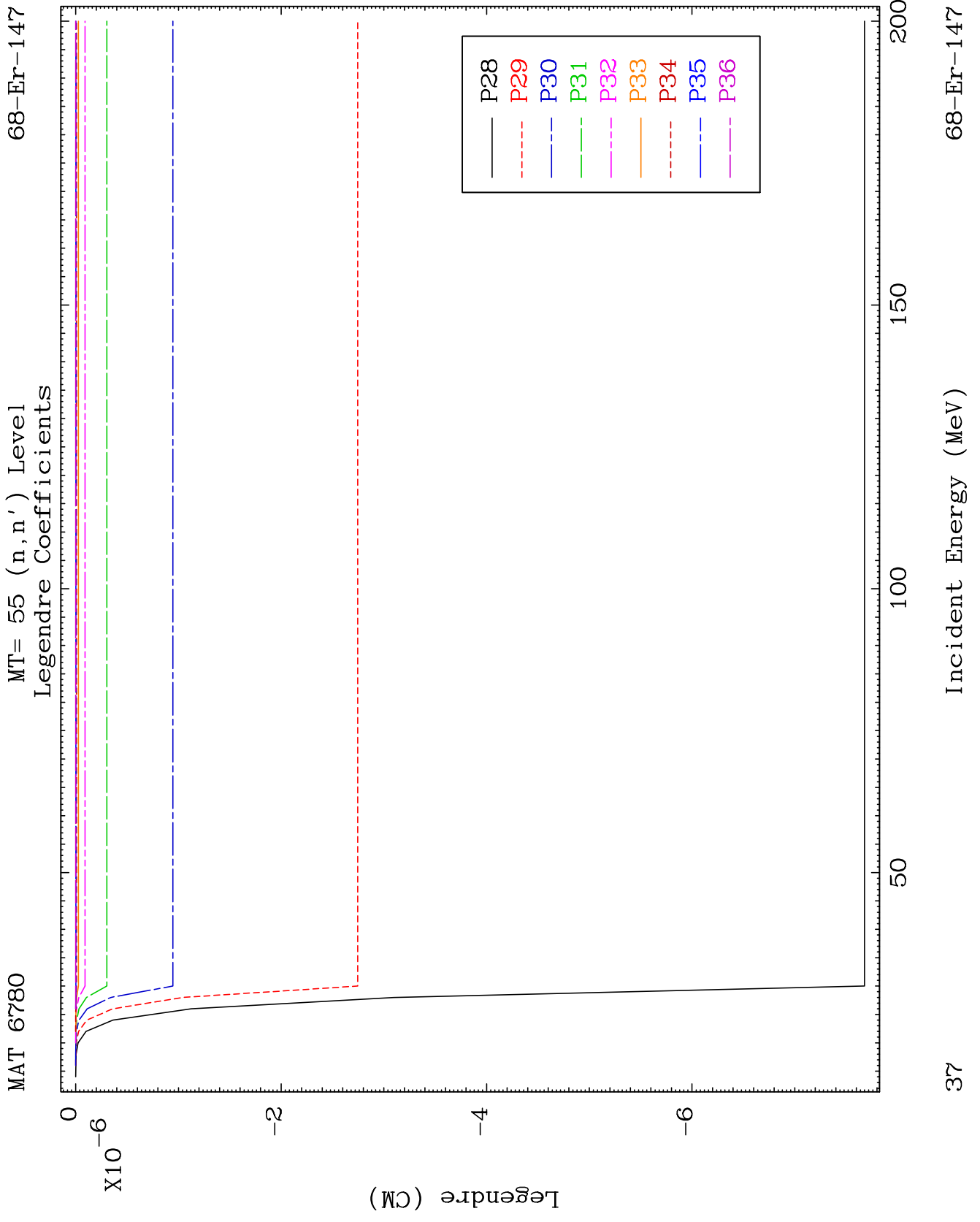


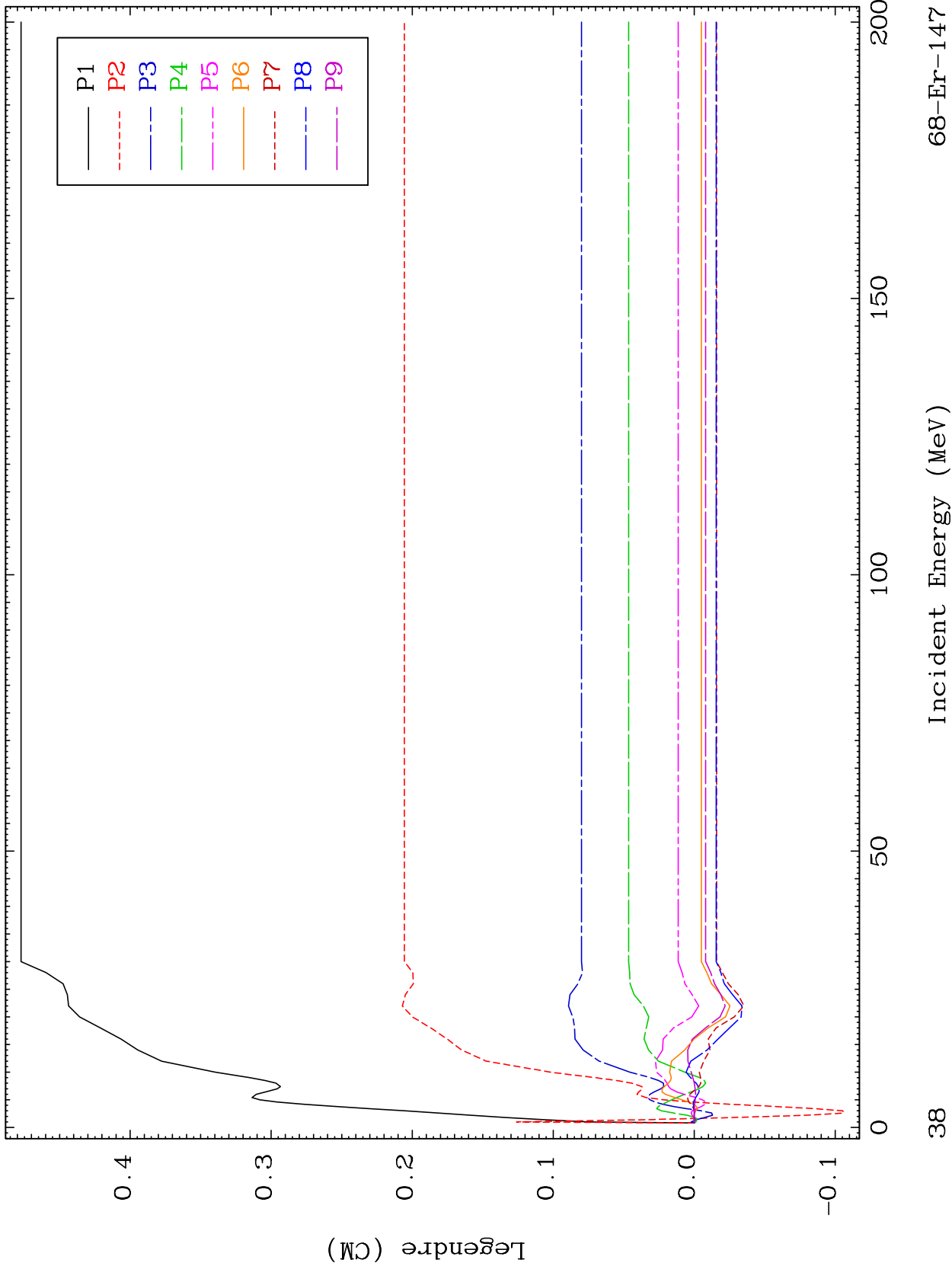
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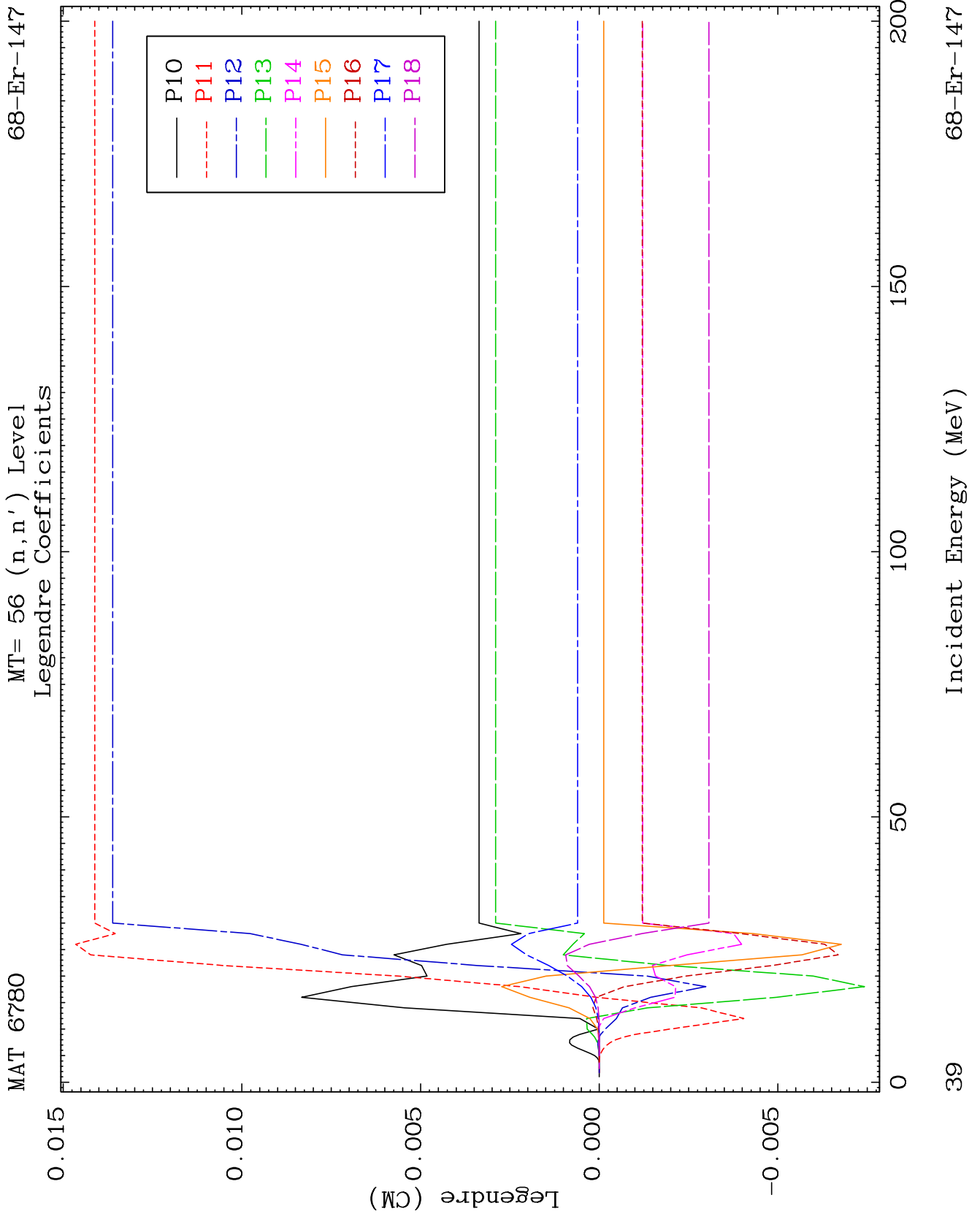
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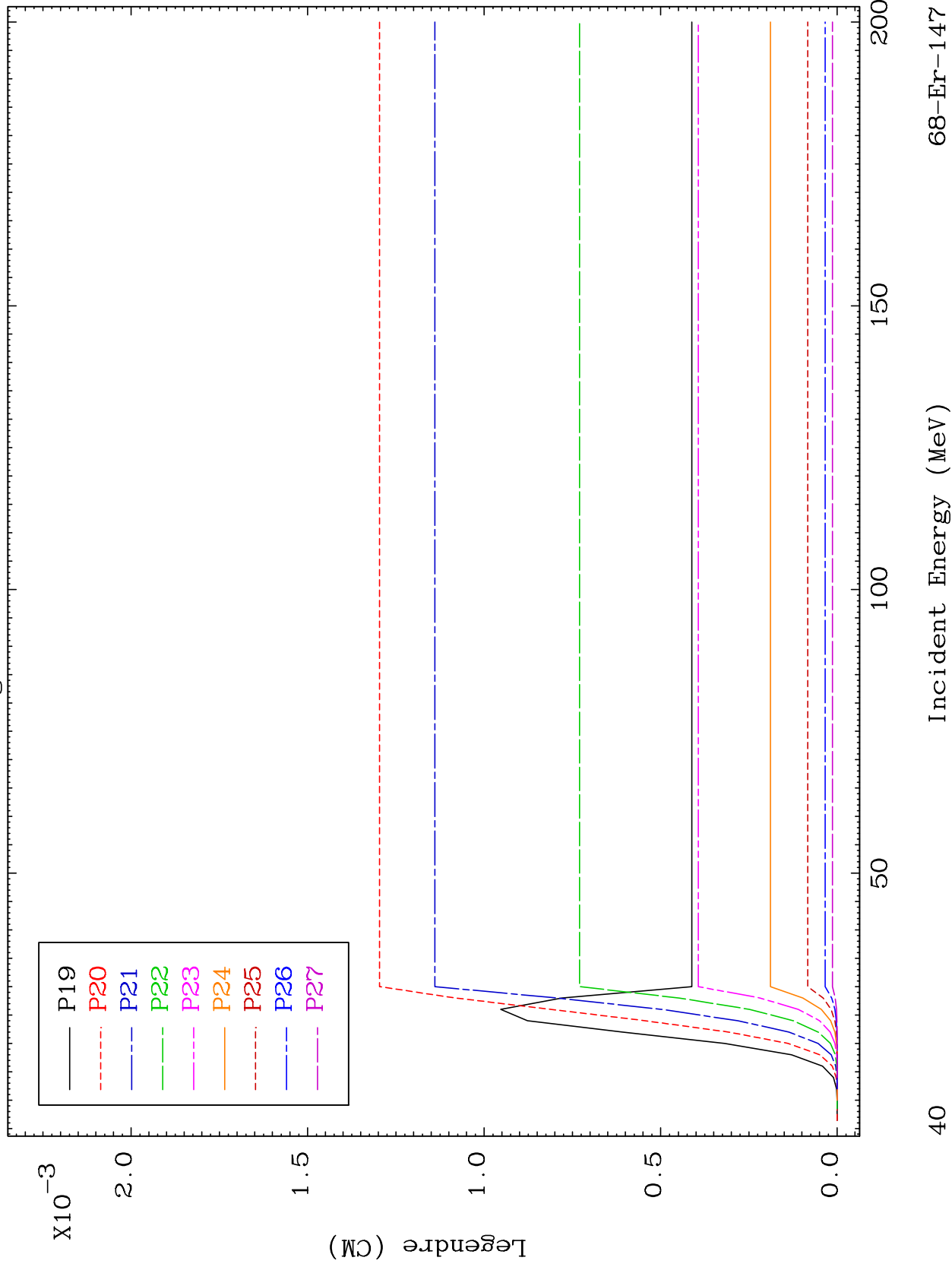




MAT 6780

MT= 56 (n,n') Level
Legendre Coefficients

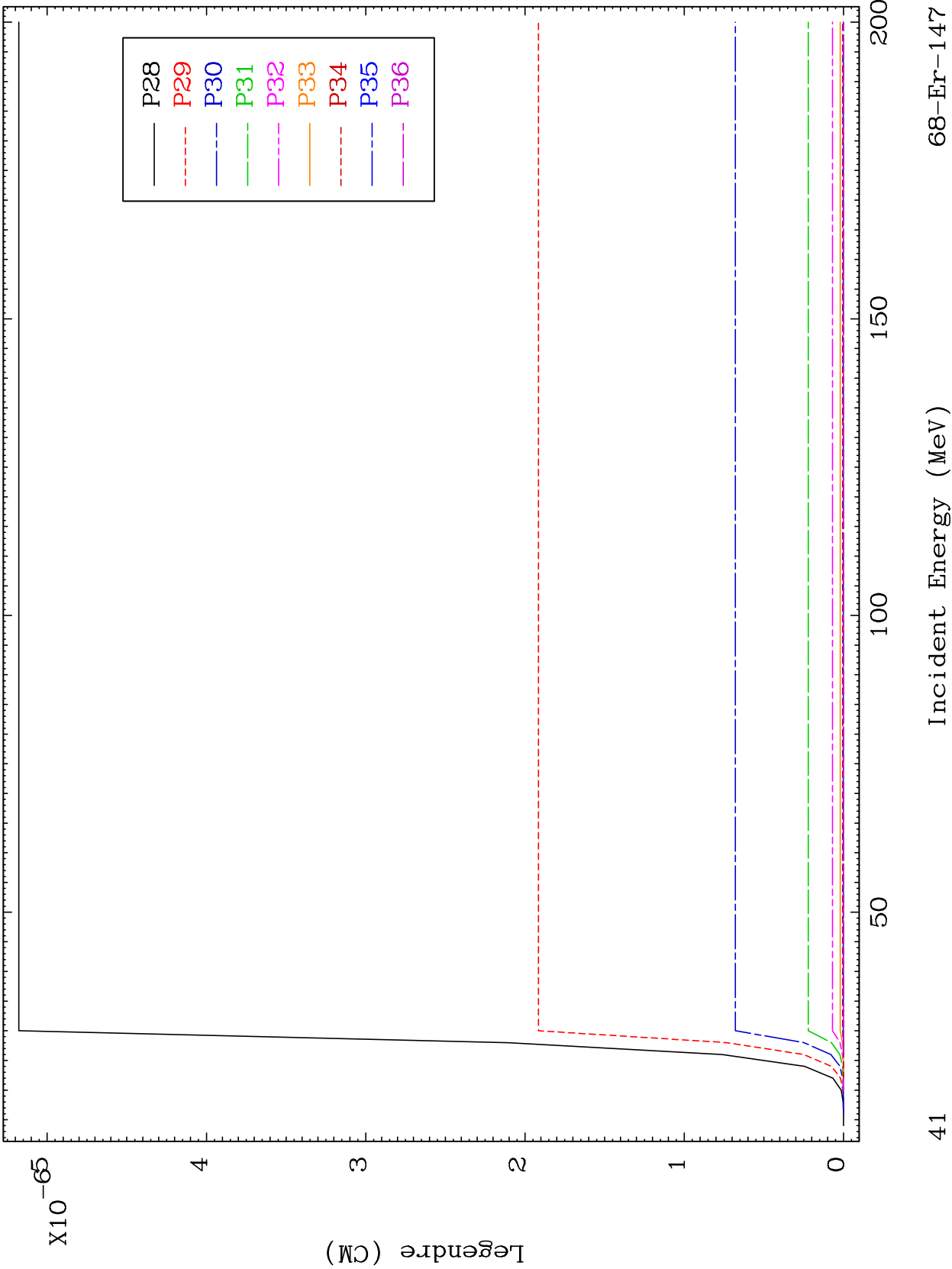
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MAT 6780

MT= 56 (n,n') Level
Legendre Coefficients

68-Er-147



41

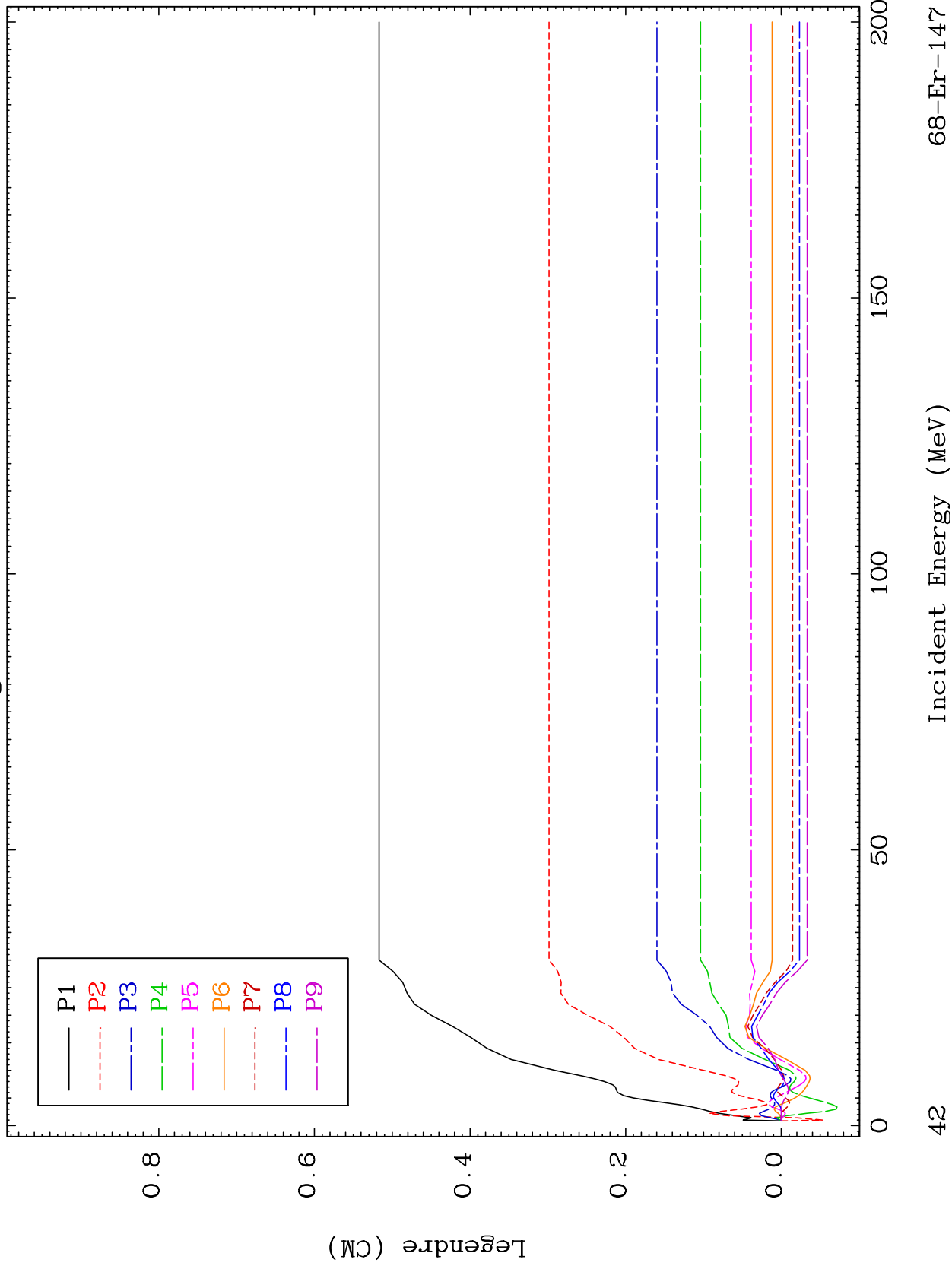
Incident Energy (MeV)

68-Er-147

MAT 6780

MT= 57 (n,n') Level
Legendre Coefficients

68-Er-147



68-Er-147

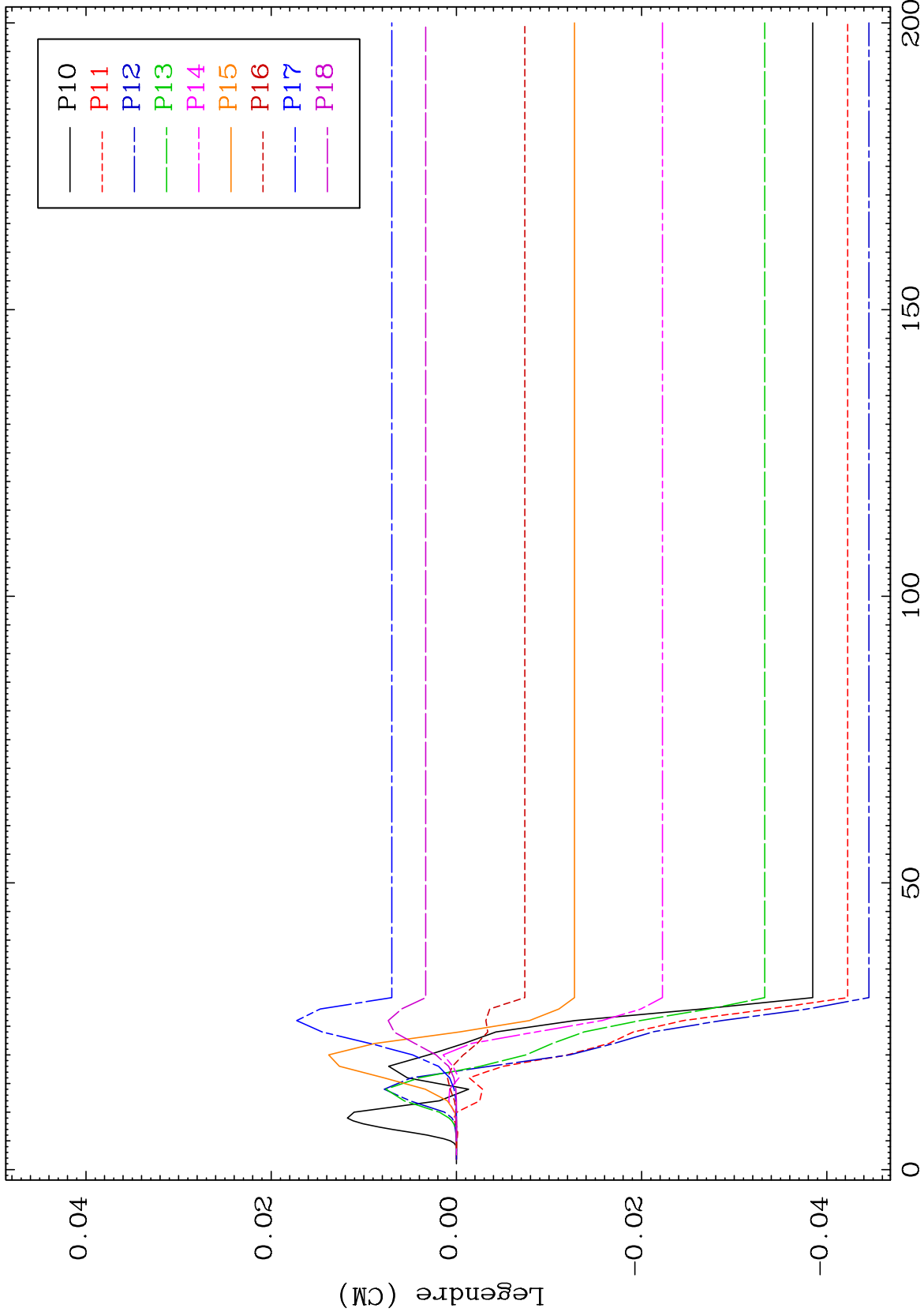
Incident Energy (MeV)

42

MAT 6780

MT= 57 (n,n') Level
Legendre Coefficients

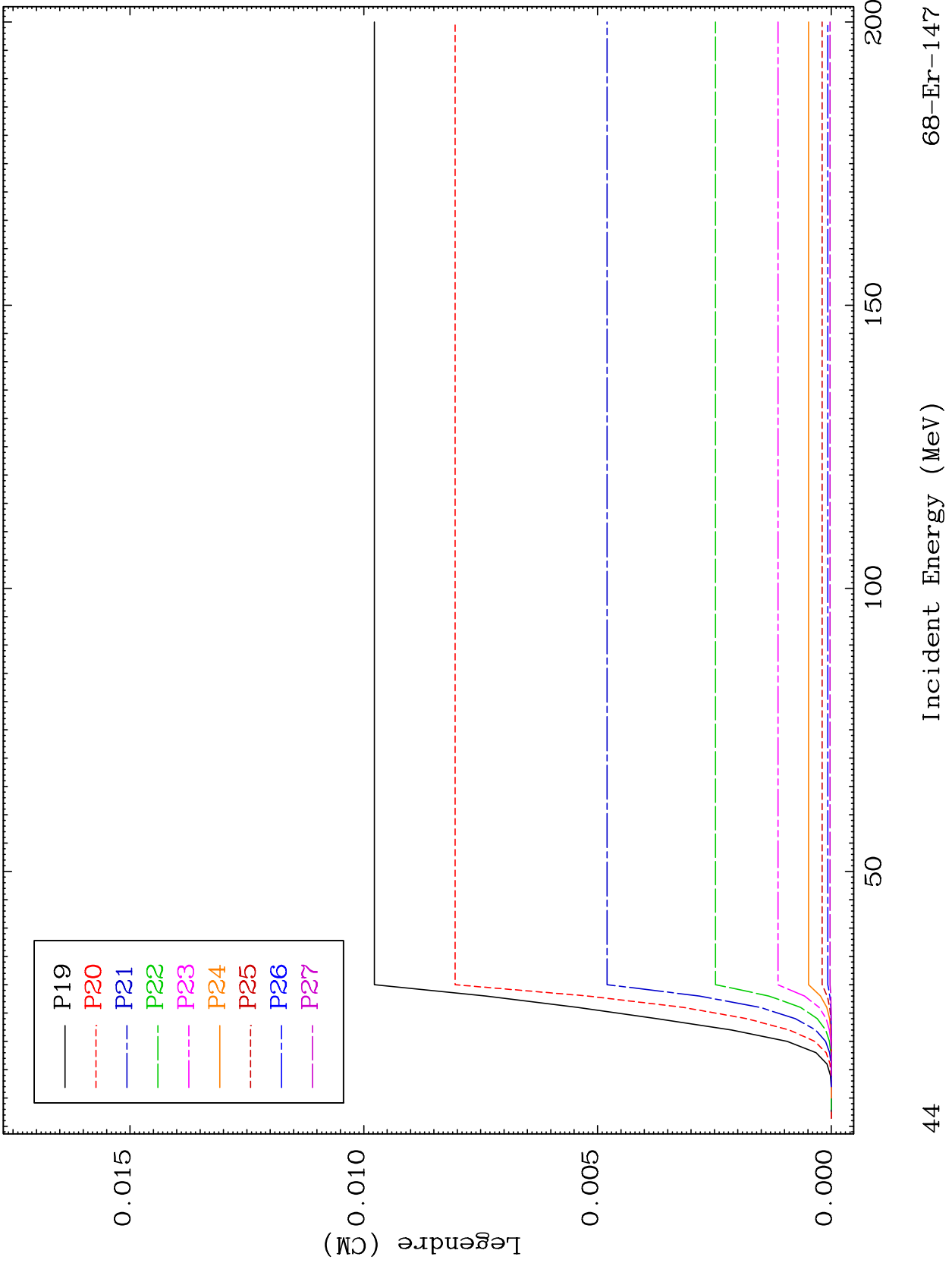
68-Er-147



43

Incident Energy (MeV)

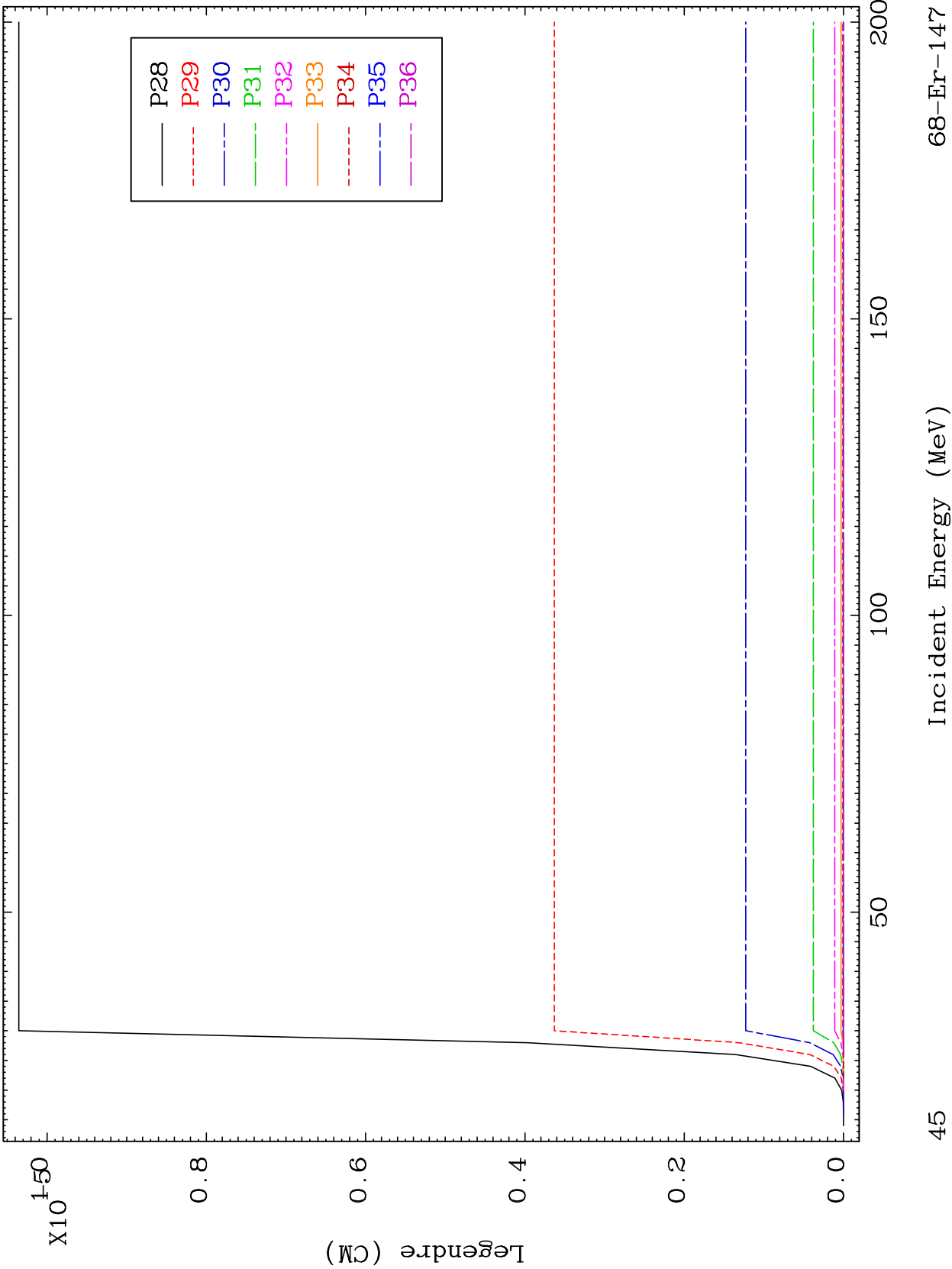
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MAT 6780

MT= 57 (n,n') Level
Legendre Coefficients

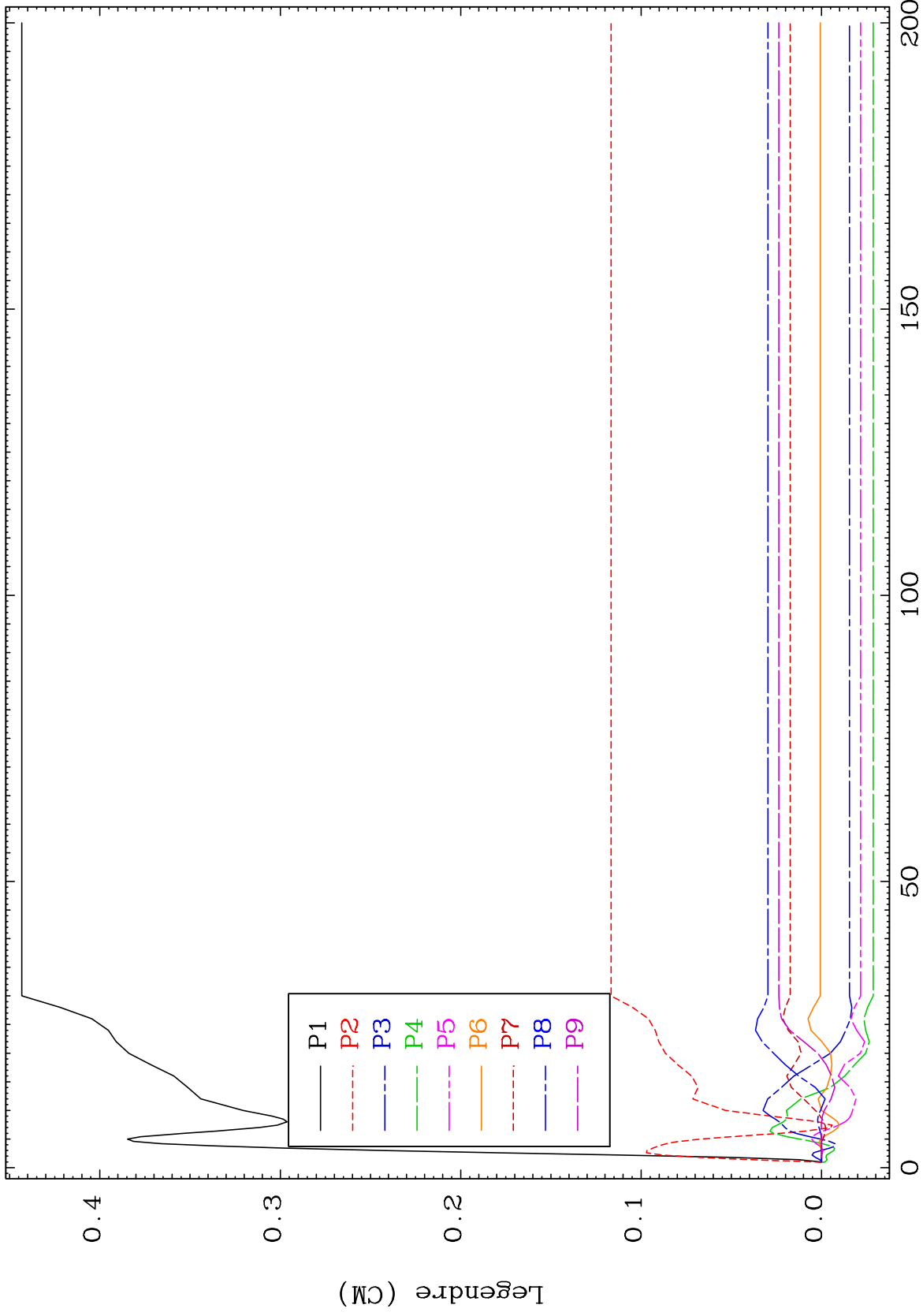
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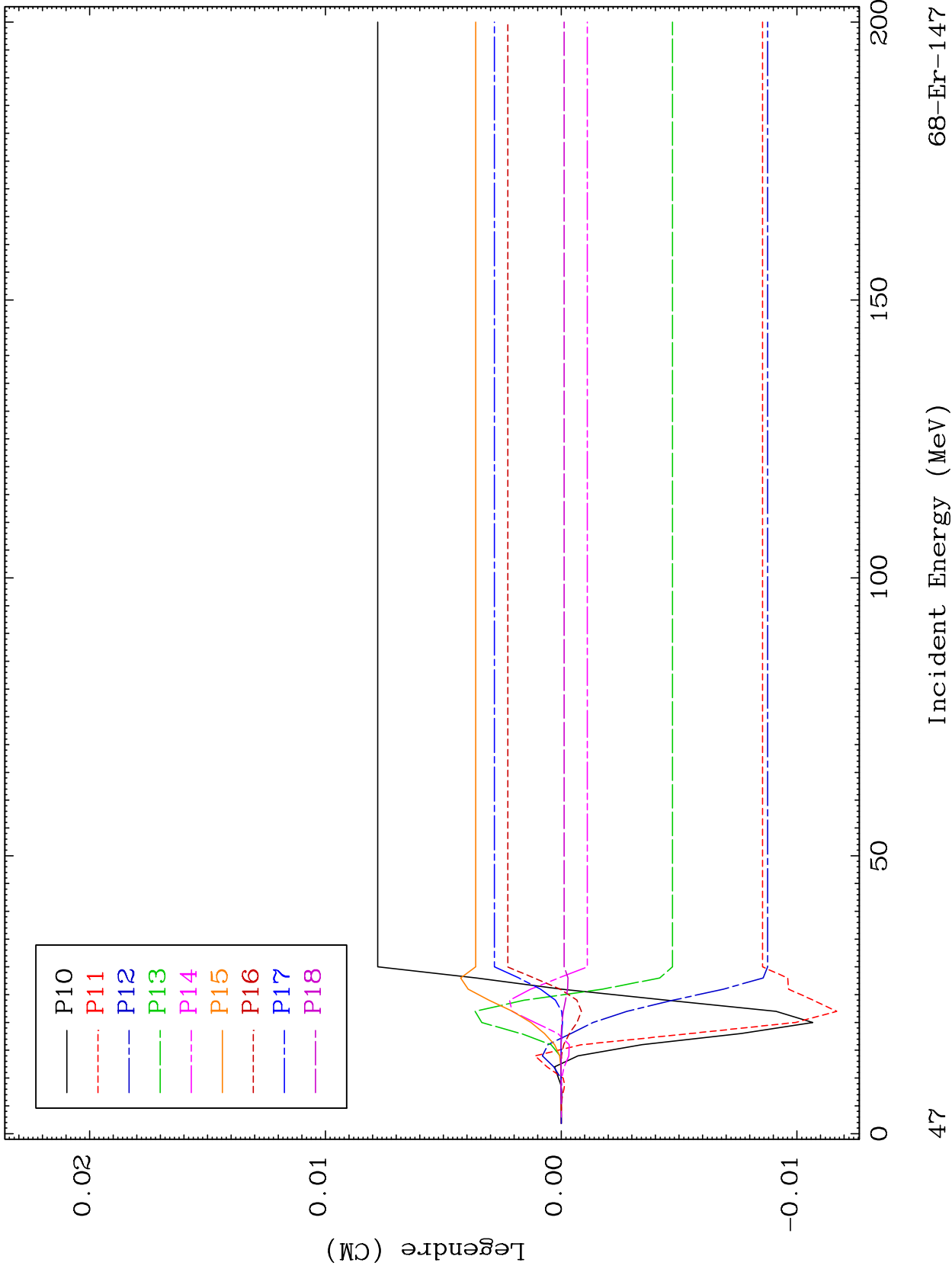


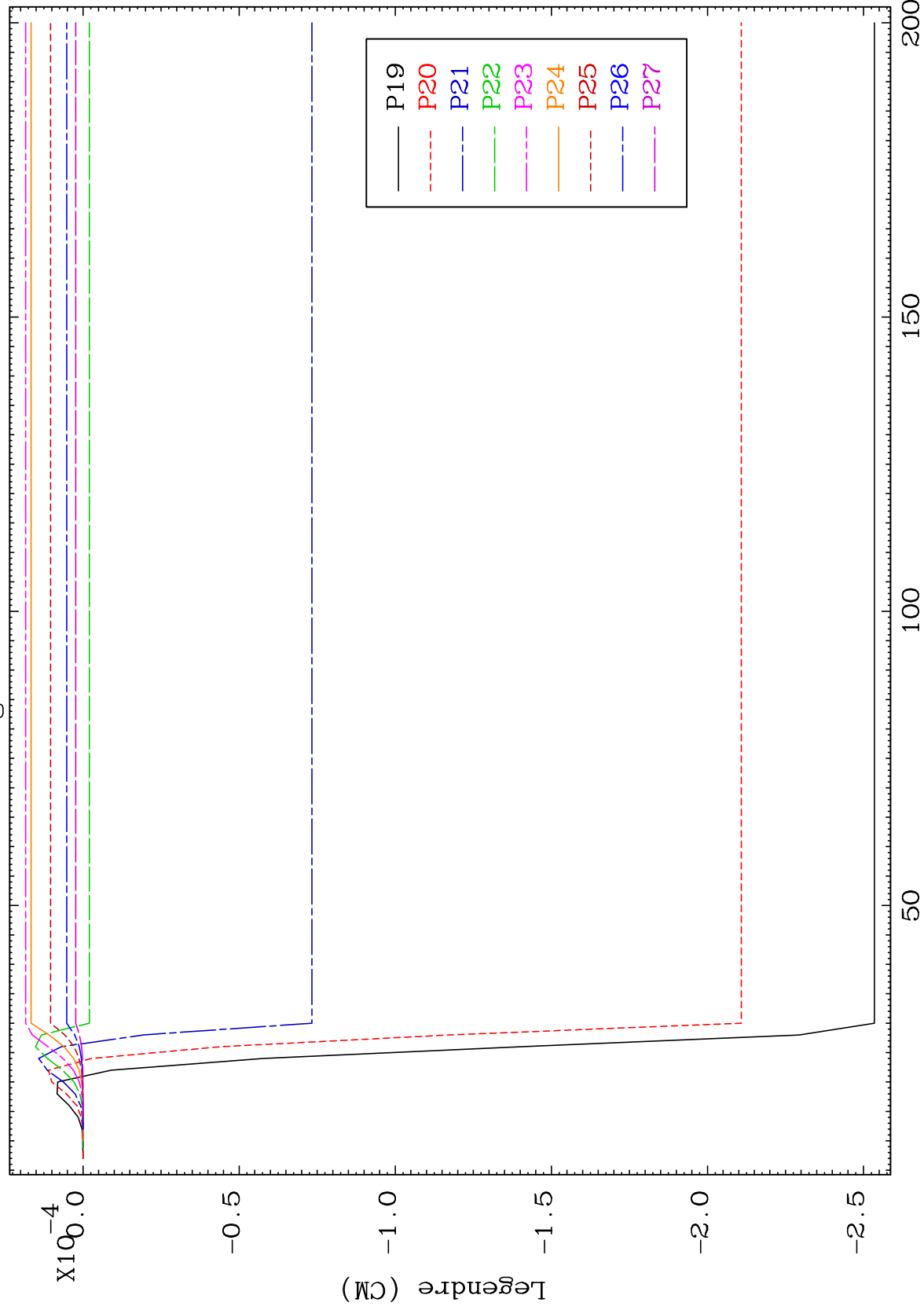
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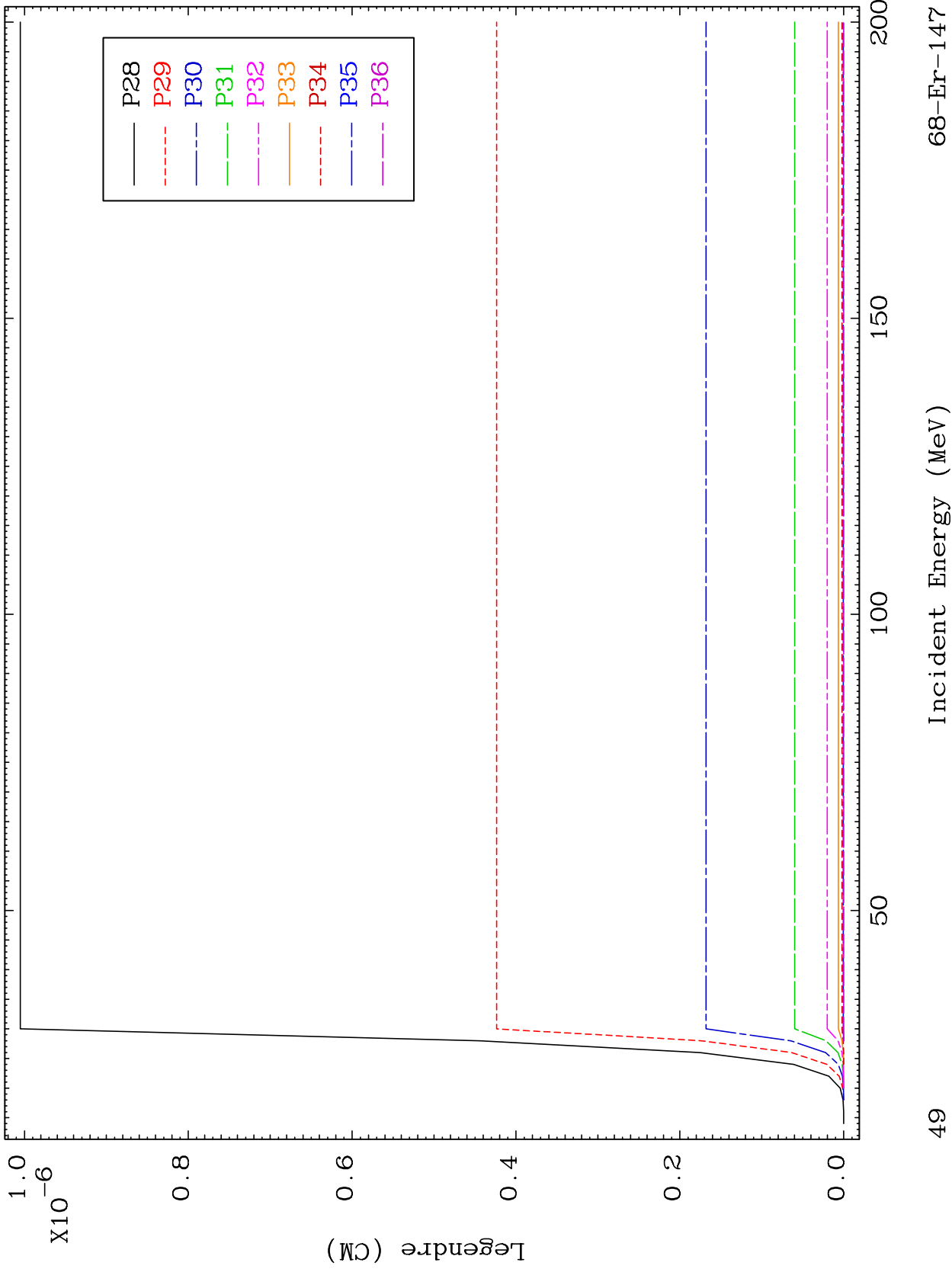
Incident Energy (MeV)

45





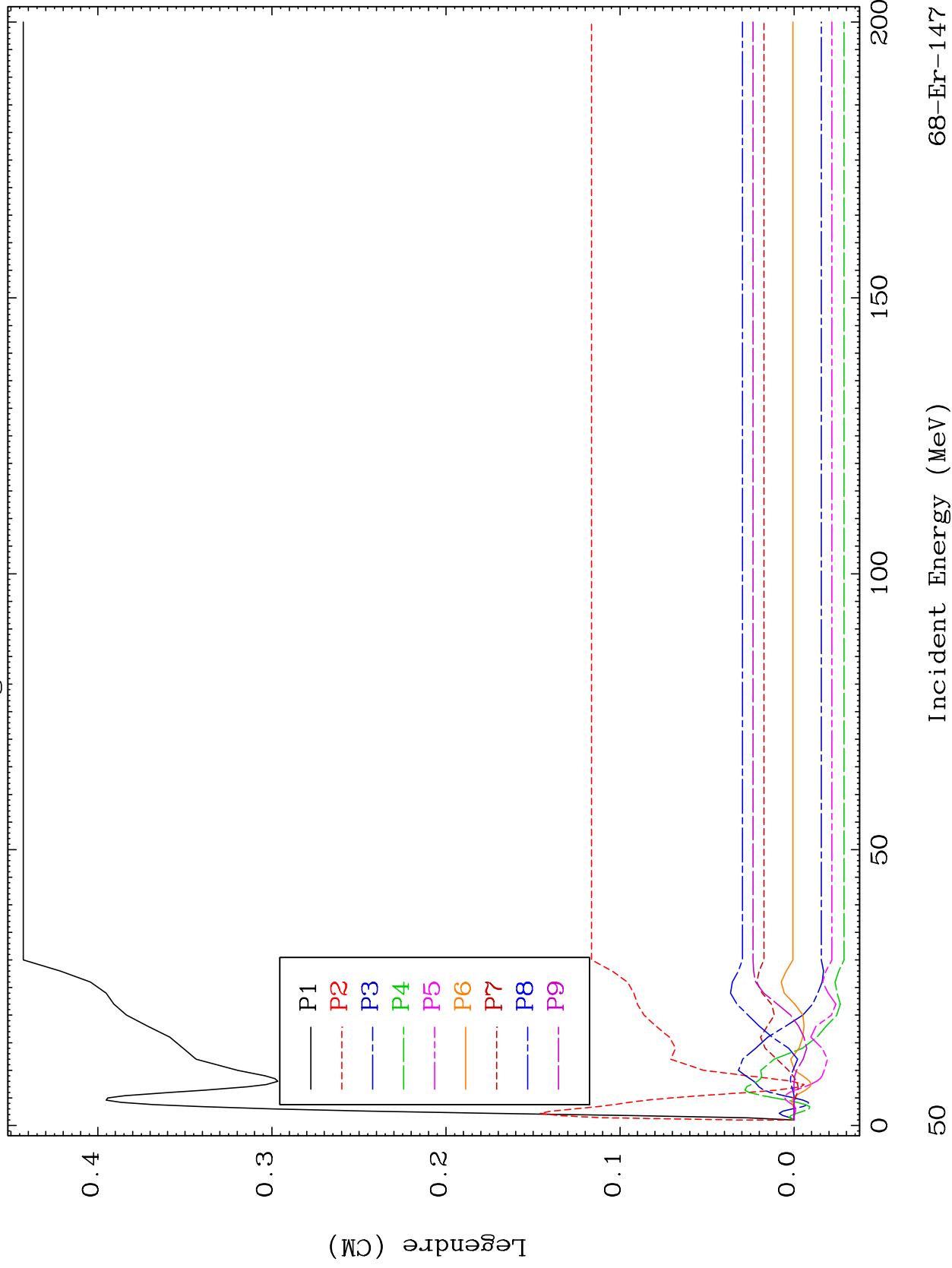




MAT 6780

MT= 59 (n,n') Level
Legendre Coefficients

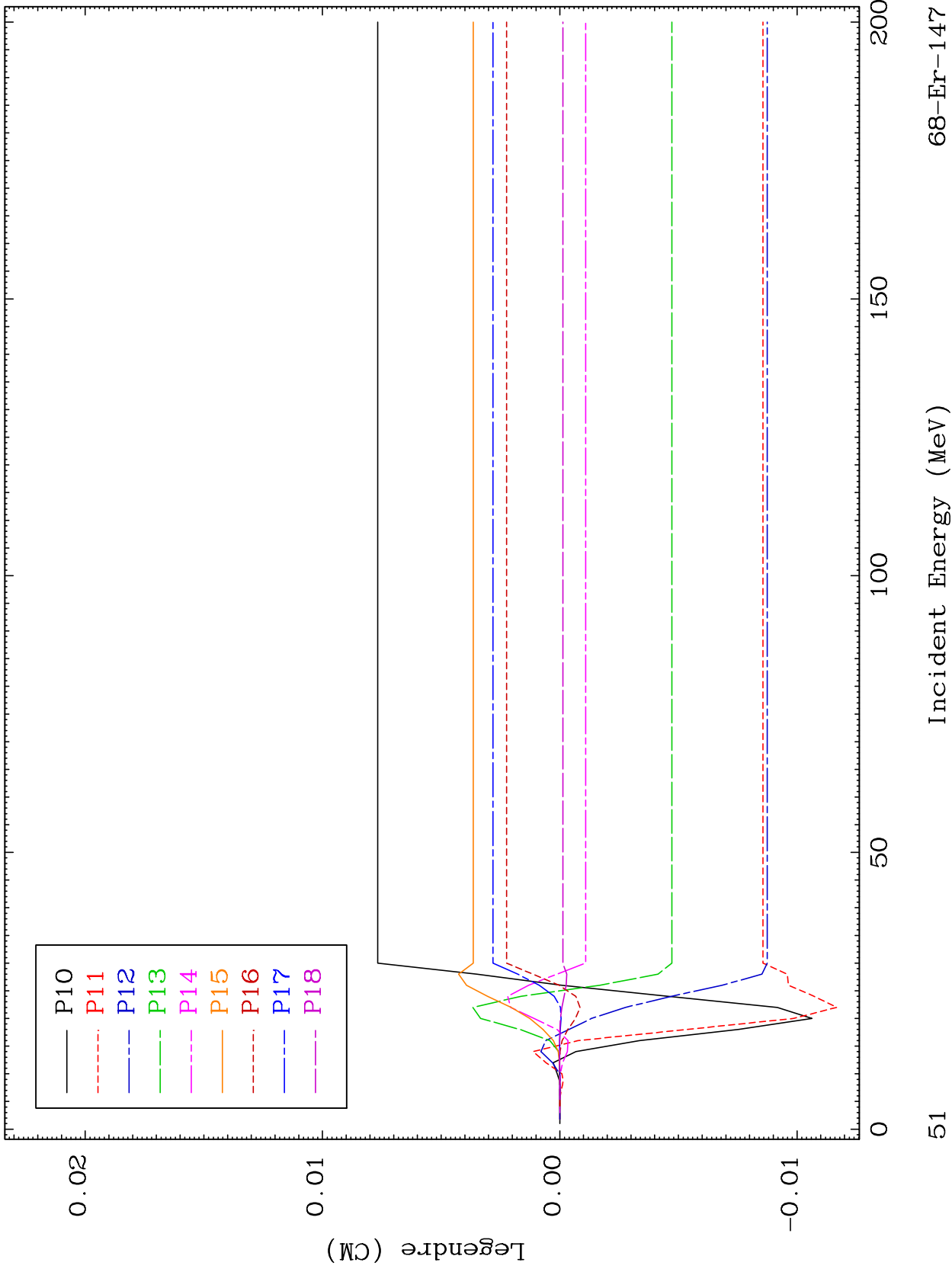
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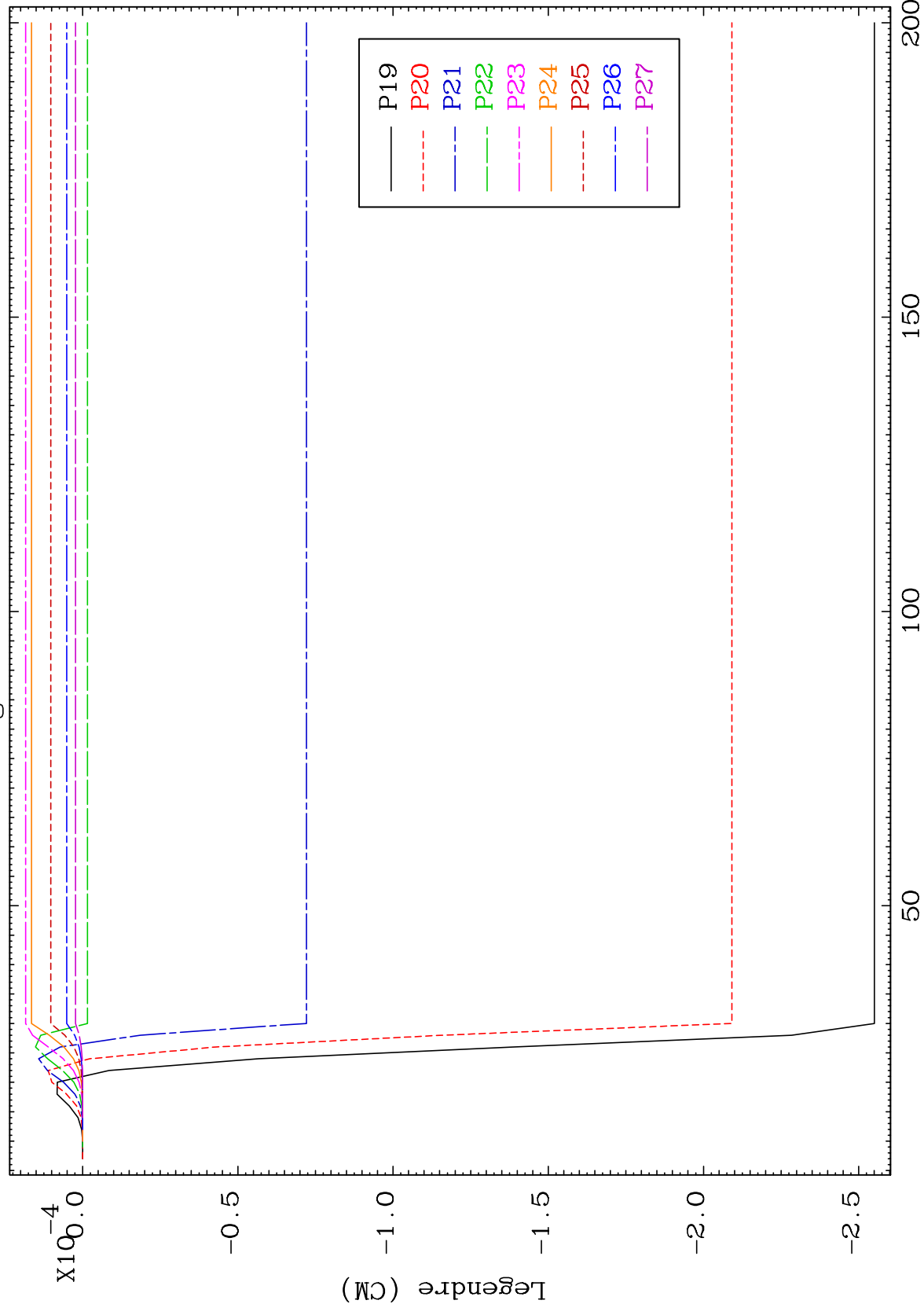


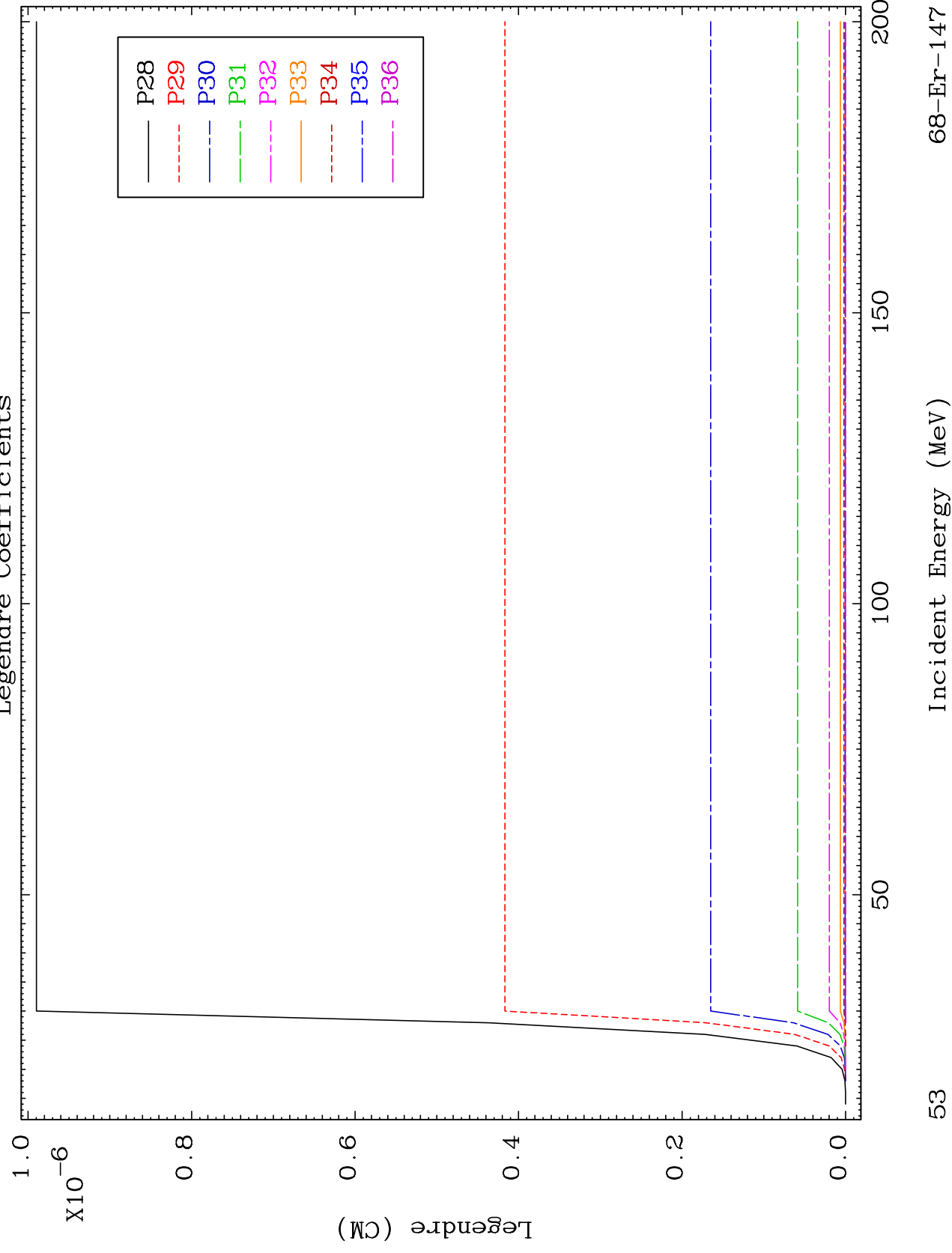
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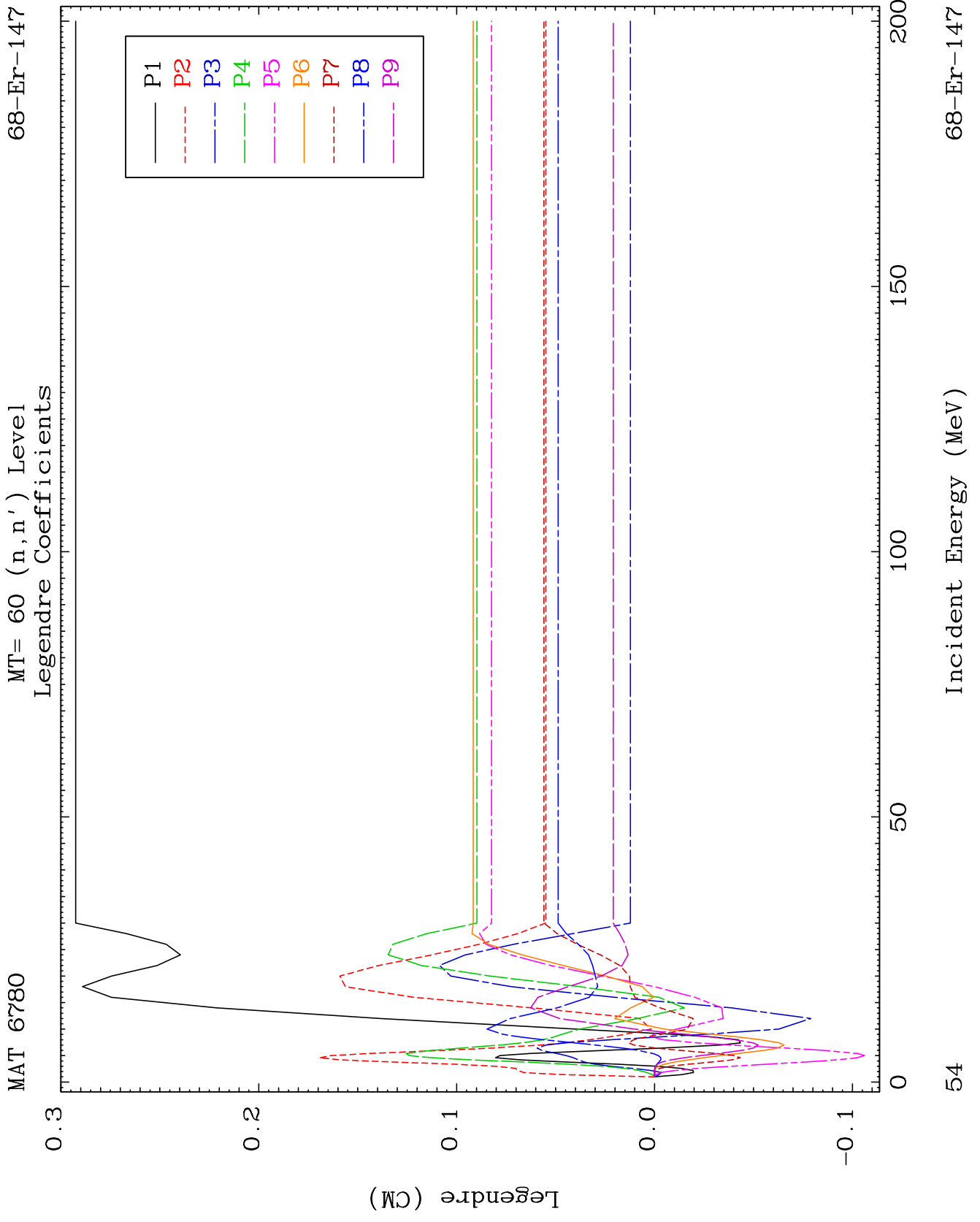
Incident Energy (MeV)

50





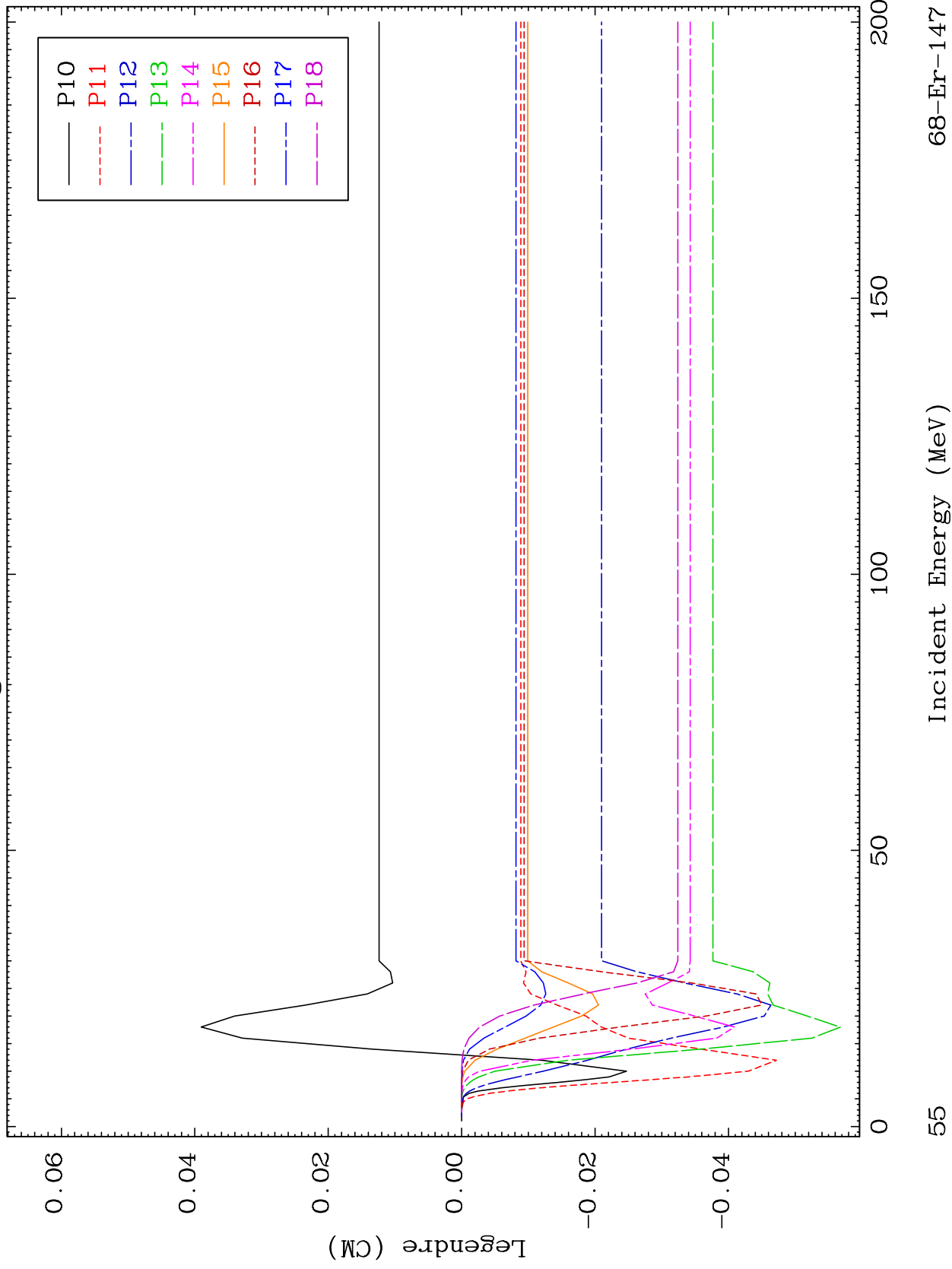




MAT 6780

MT= 60 (n,n') Level
Legendre Coefficients

68-Er-147



68-Er-147

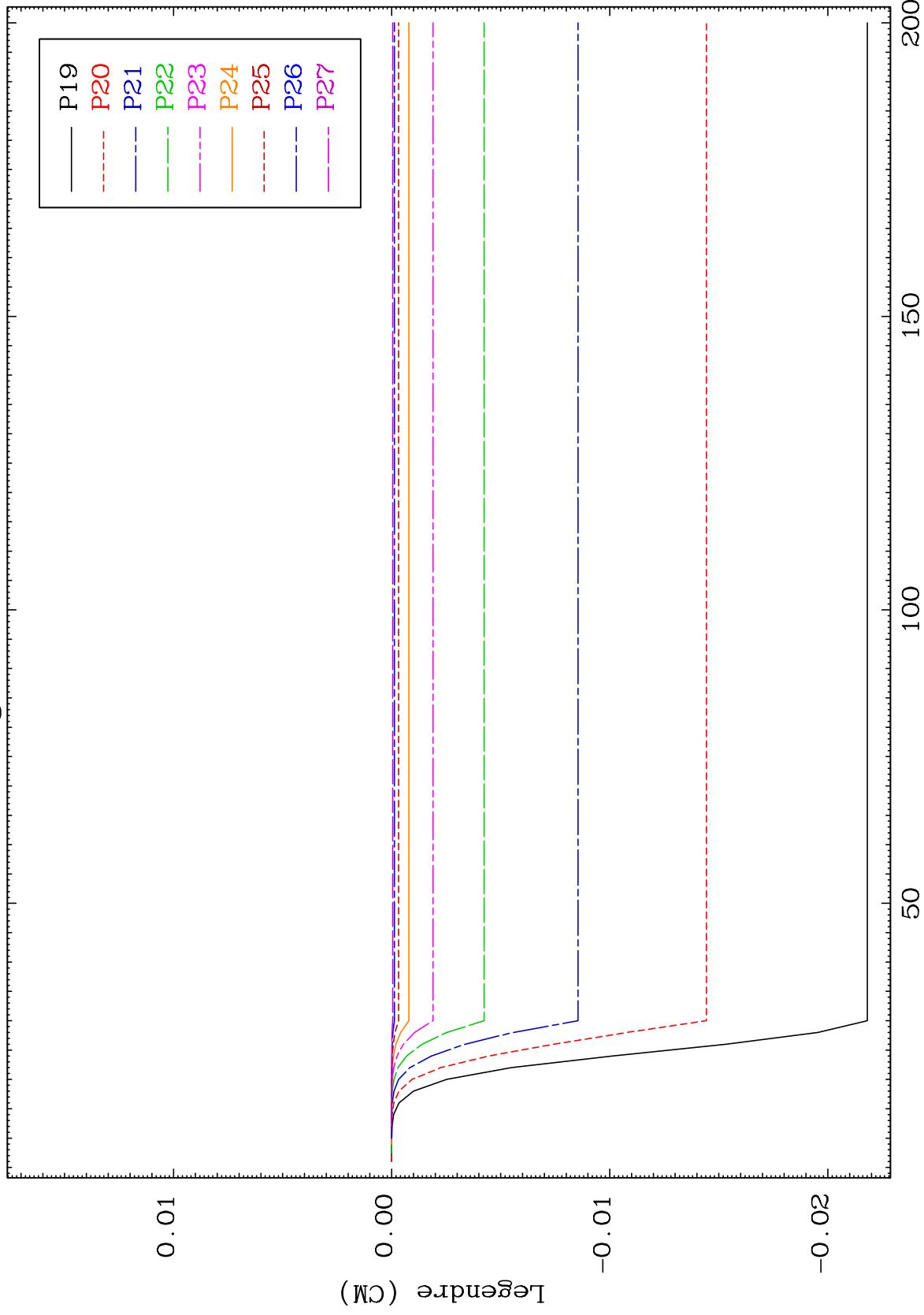
Incident Energy (MeV)

55

MAT 6780

MT= 60 (n,n') Level
Legendre Coefficients

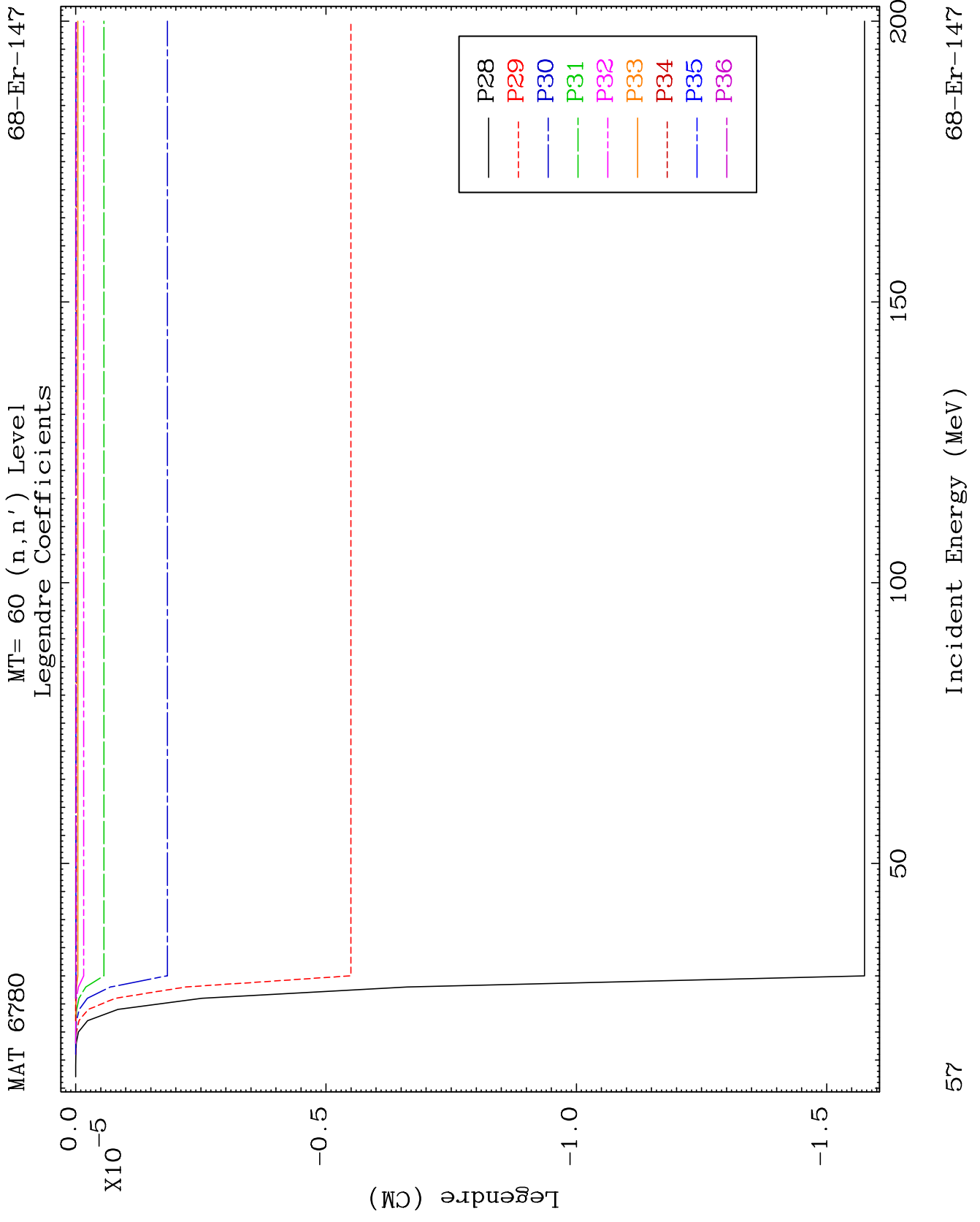
68-Er-147



56

Incident Energy (MeV)

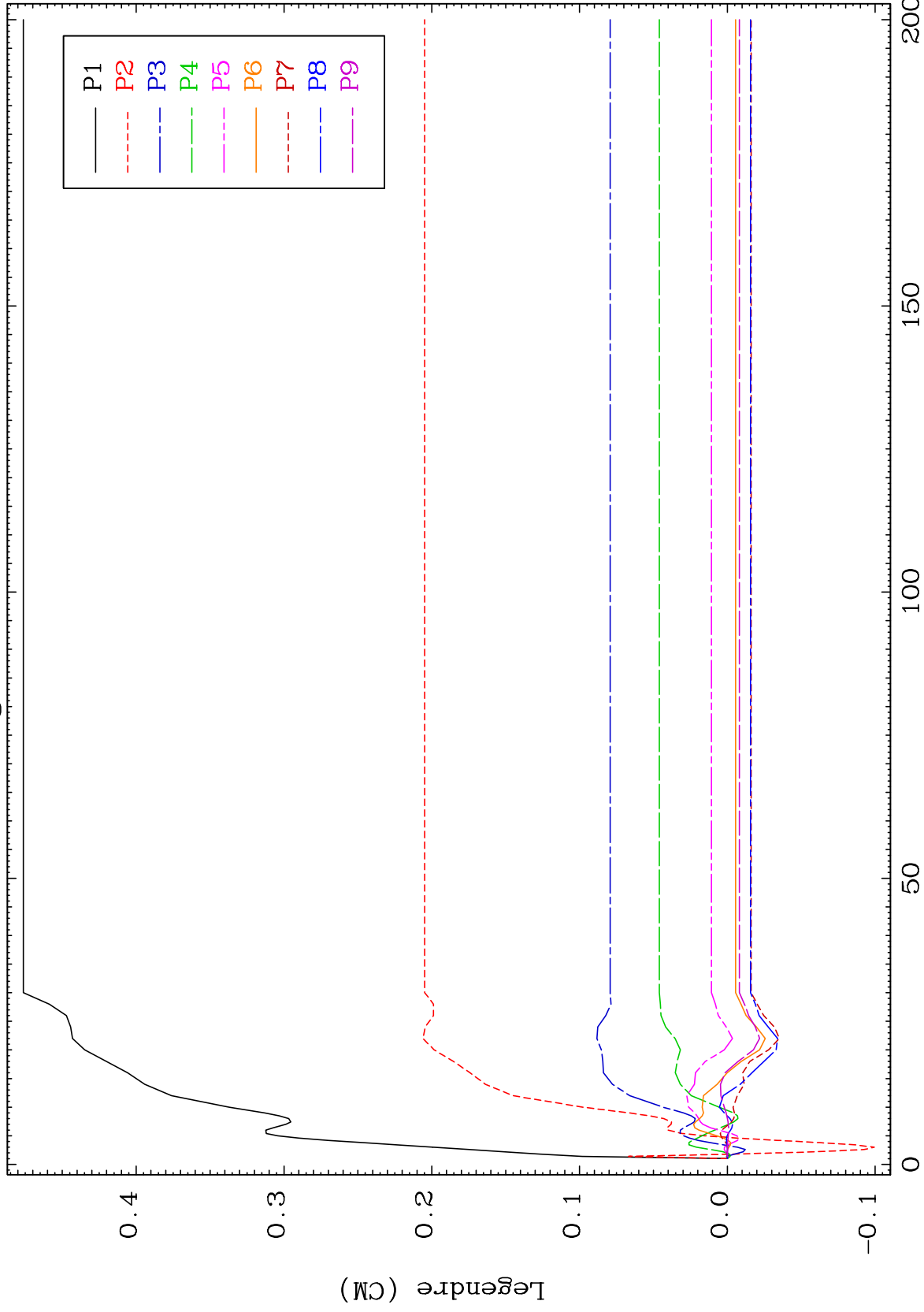
68-Er-147



MAT 6780

MT= 61 (n,n') Level
Legendre Coefficients

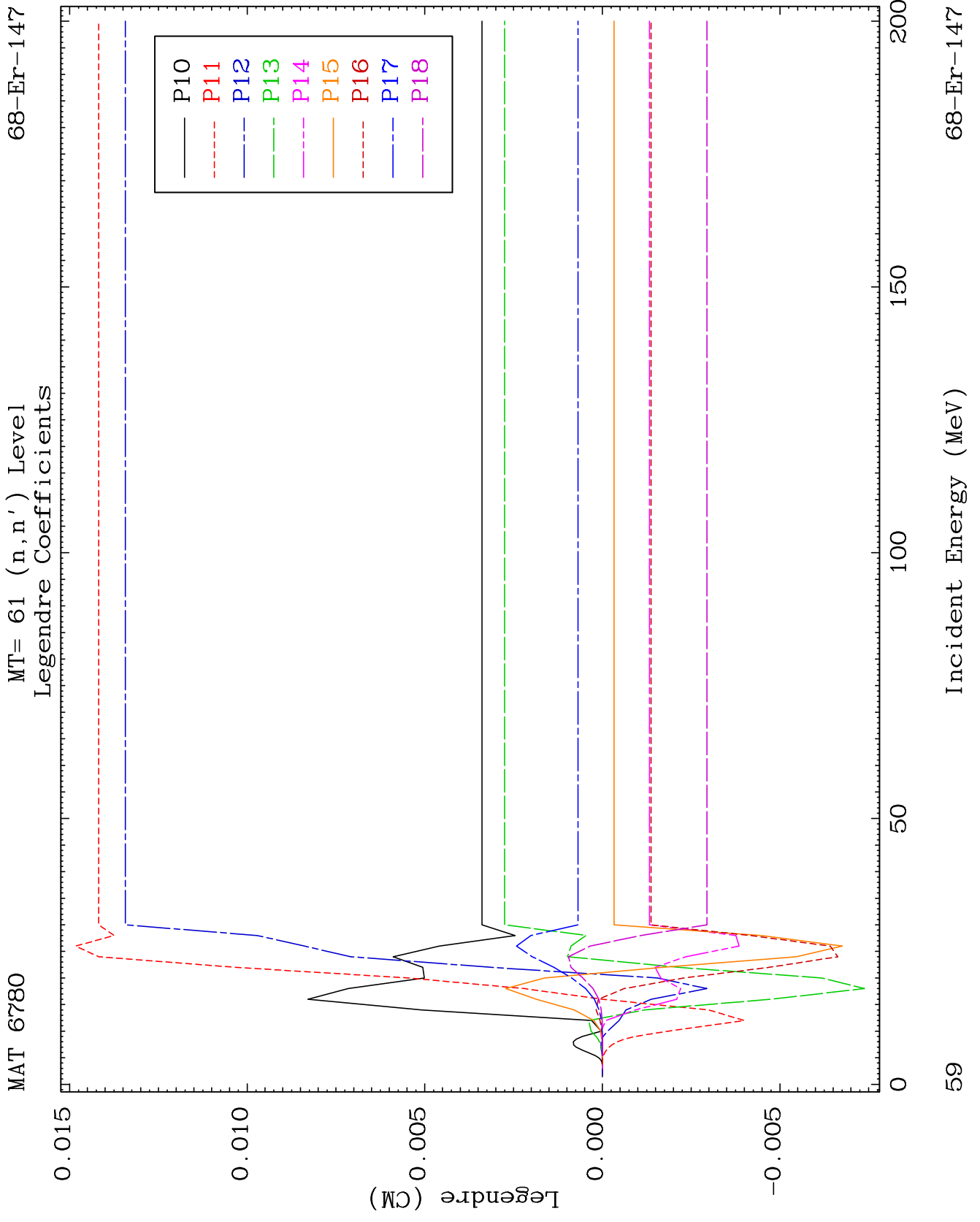
68-Er-147



68-Er-147

Incident Energy (MeV)

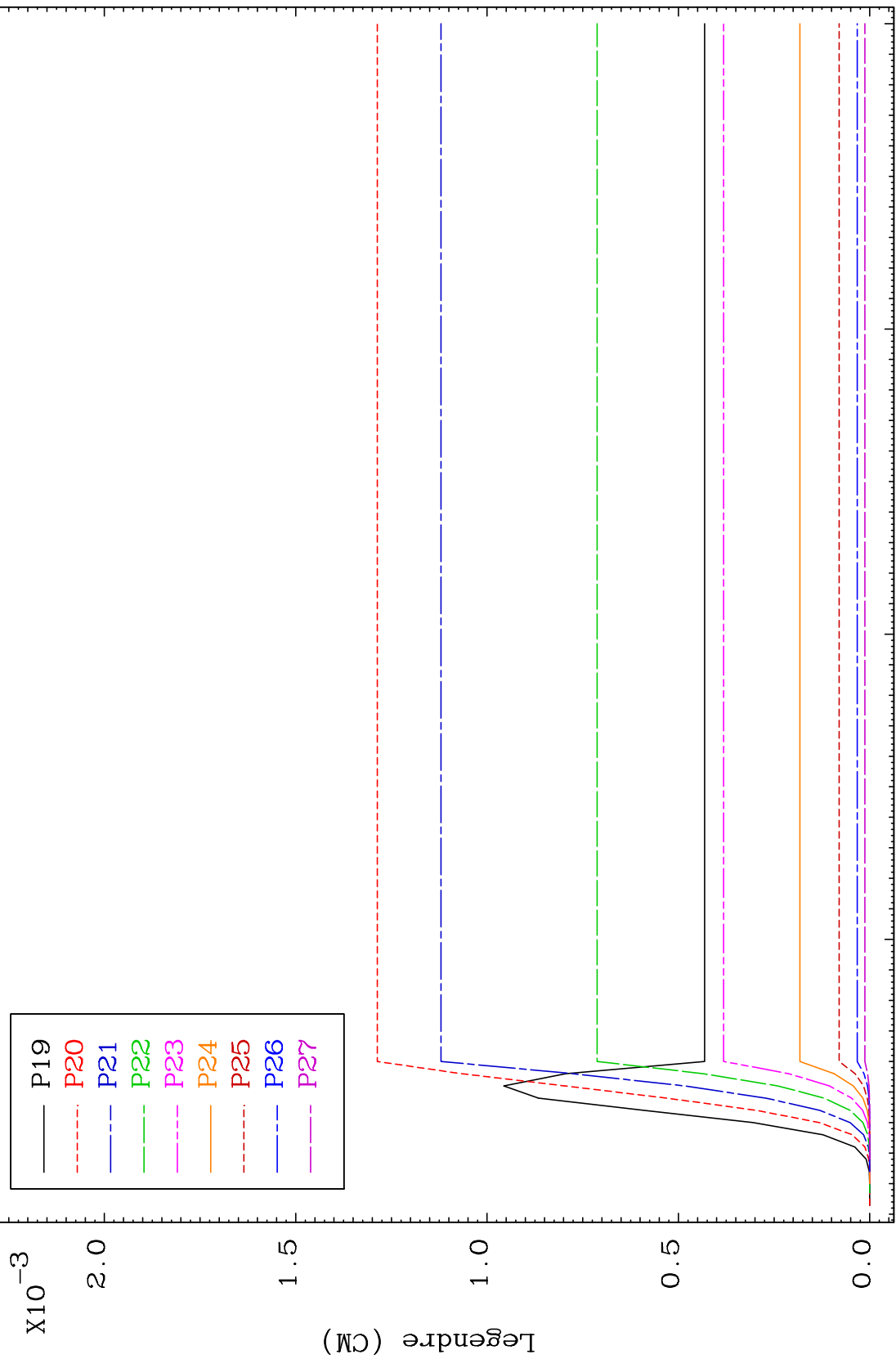
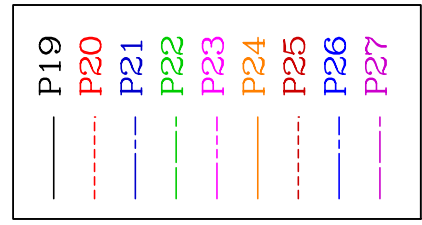
58



MAT 6780

MT= 61 (n,n') Level
Legendre Coefficients

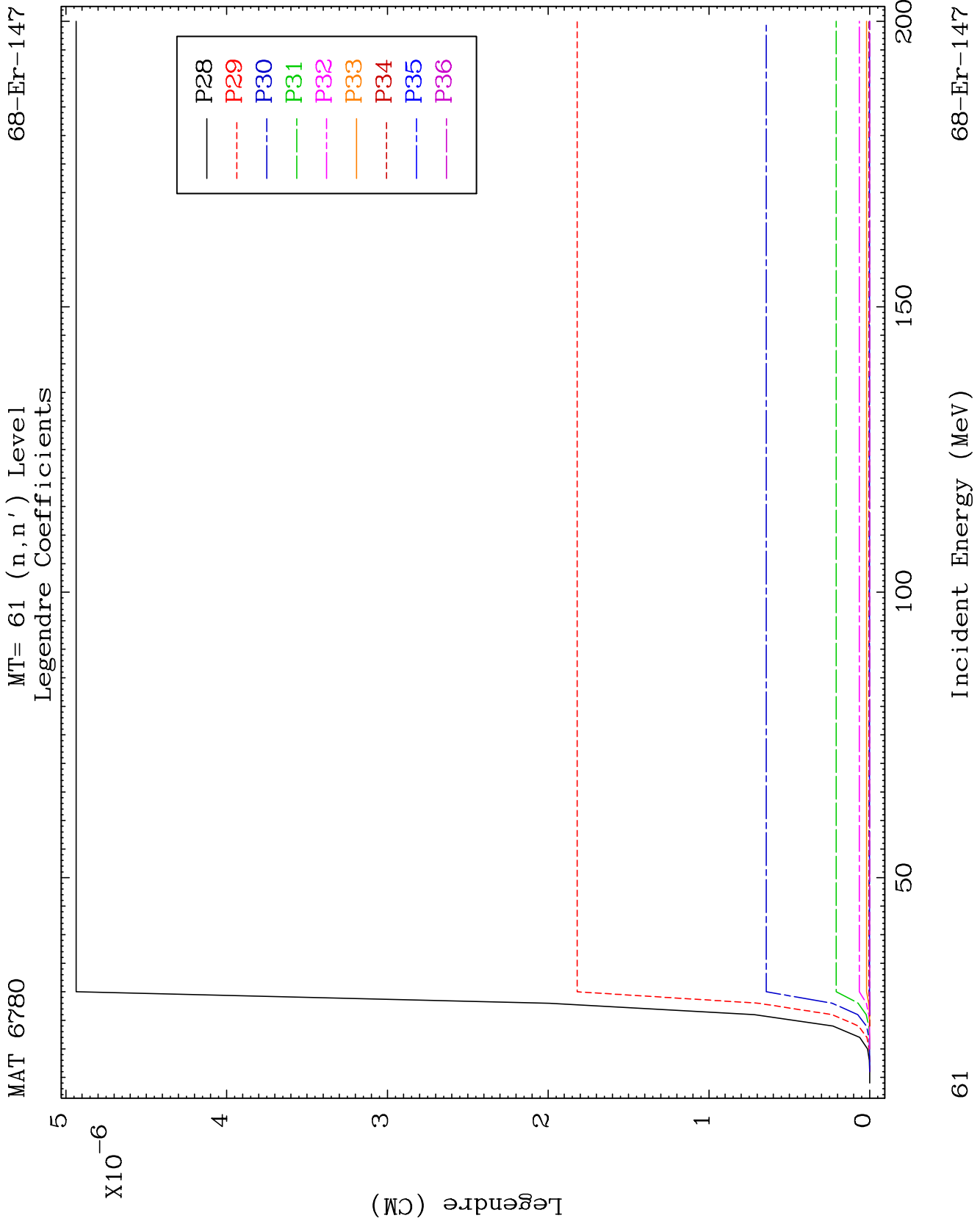
68-Er-147



60

Incident Energy (MeV)

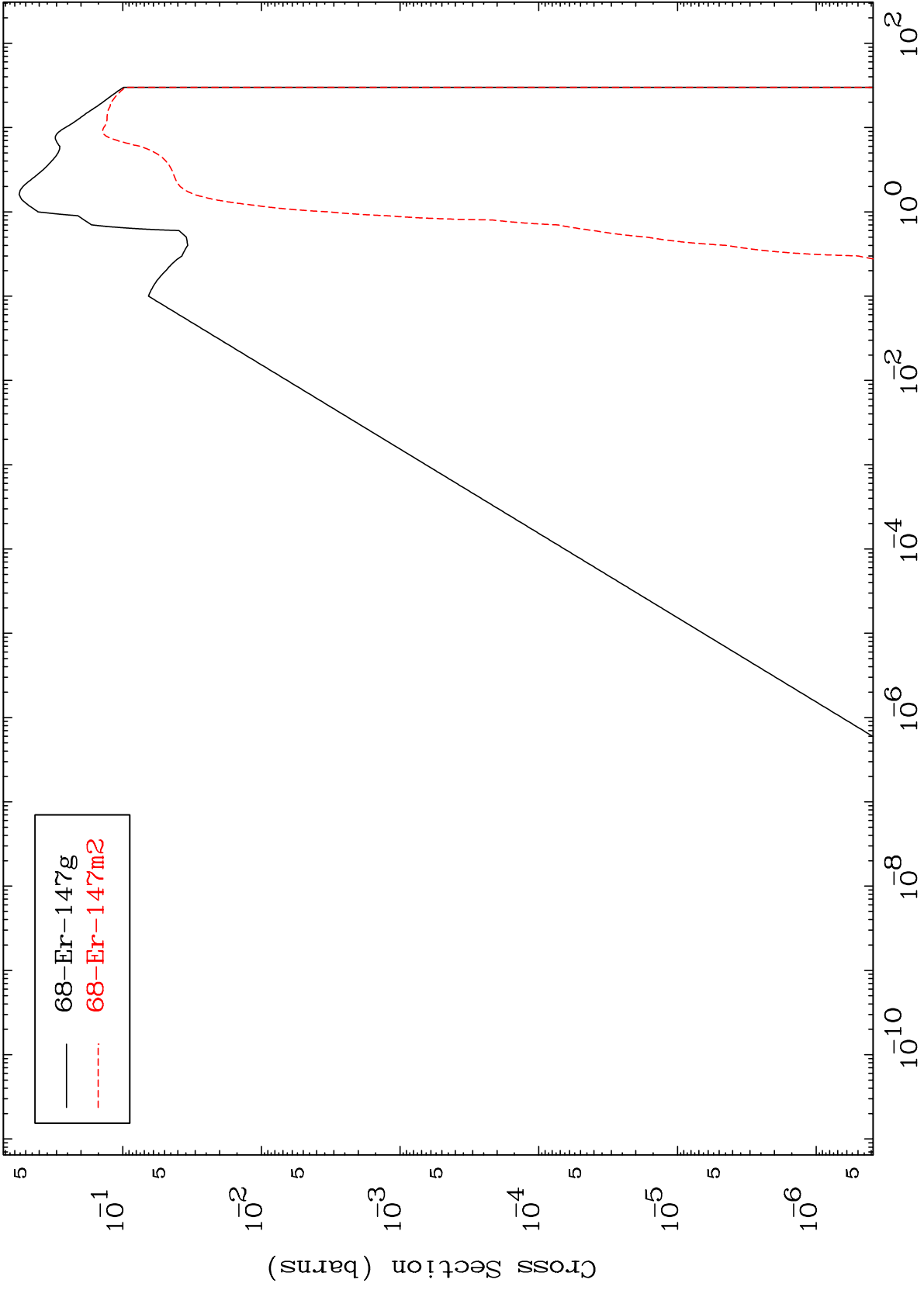
68-Er-147



MAT 6780

Inelastic
Radionuclide Production Cross Section

68-Er-147



68-Er-147g
68-Er-147m2

62

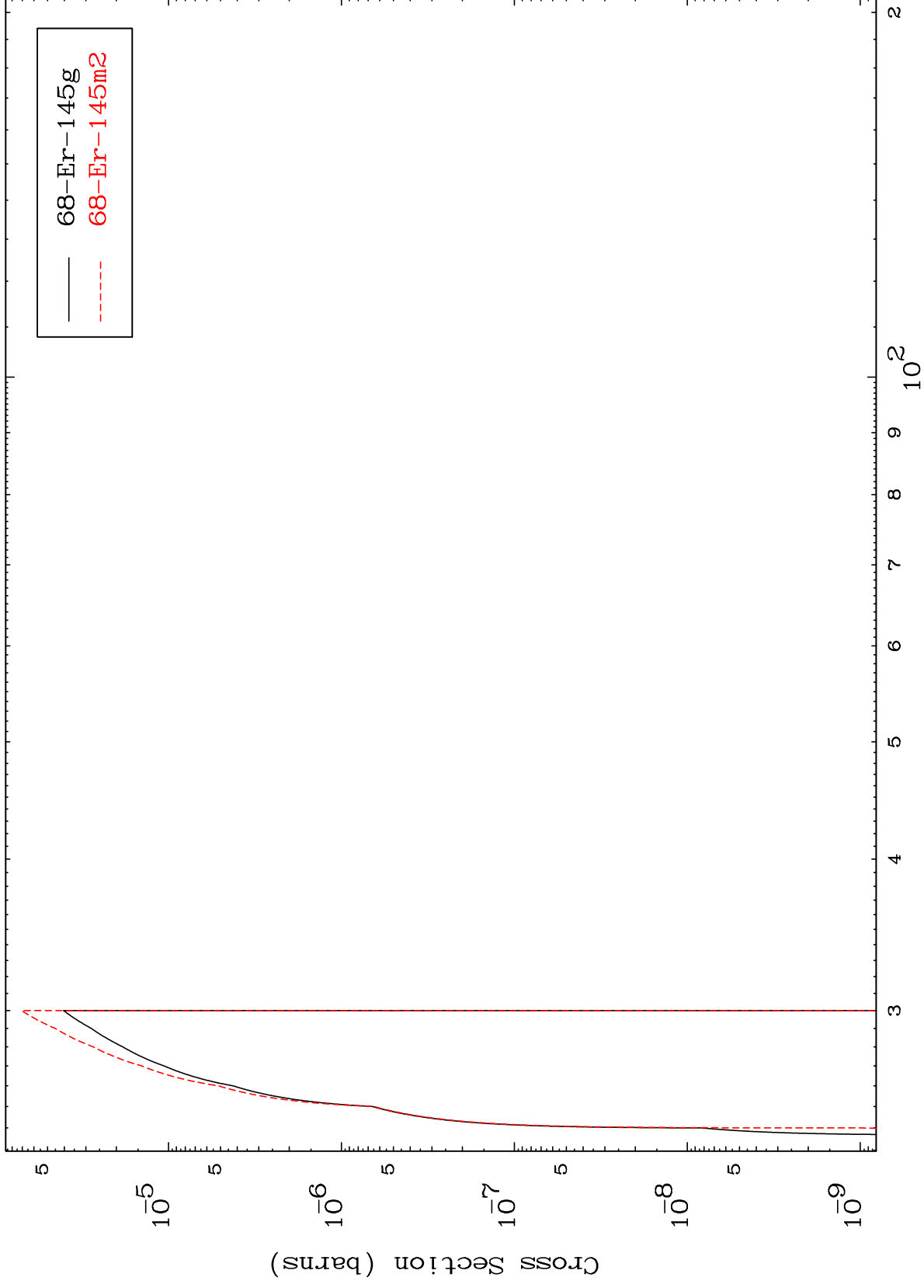
68-Er-147

MAT 6780

(n,3n)

68-Er-147

Radionuclide Production Cross Section



63

Incident Energy (MeV)

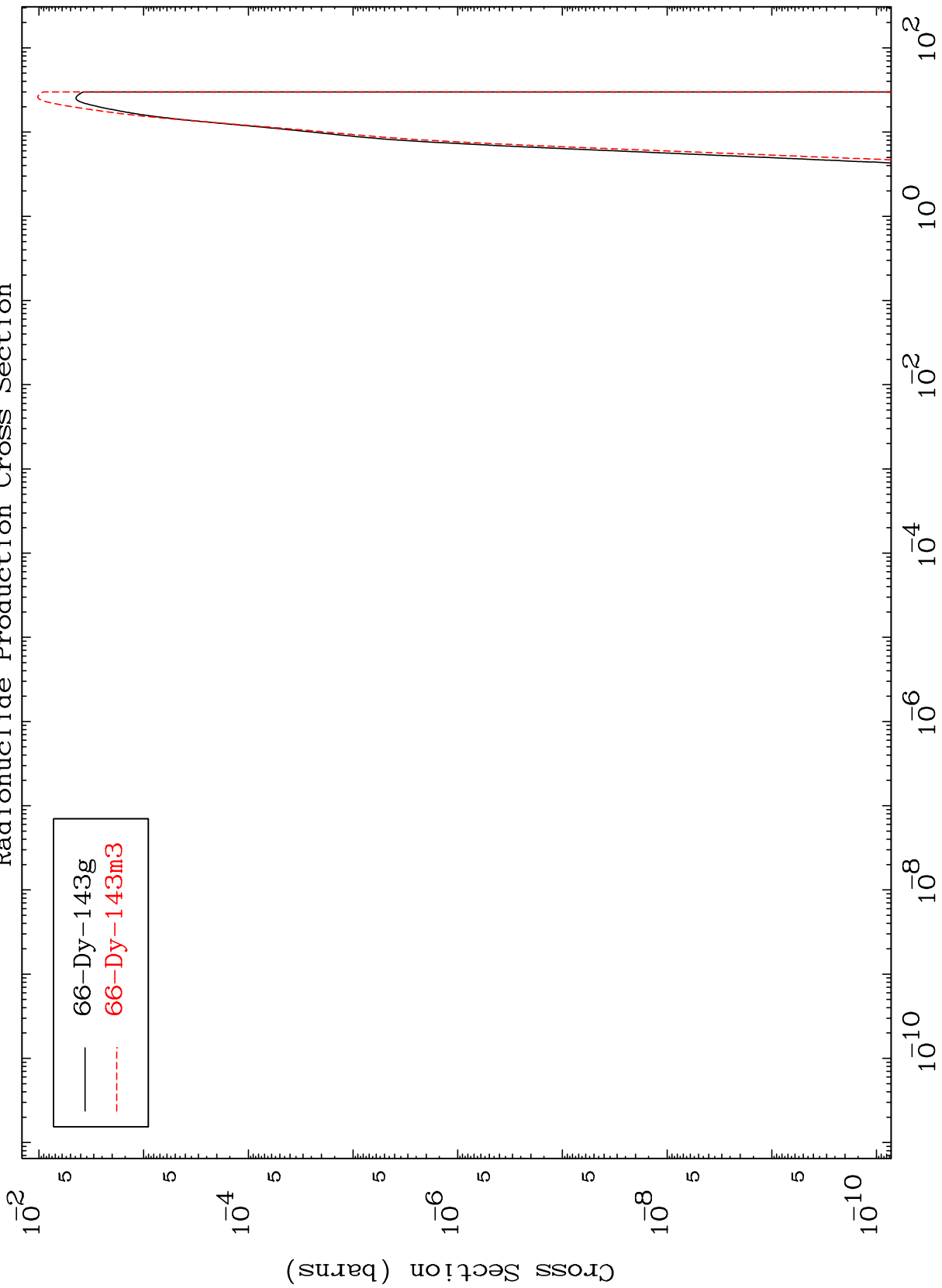
68-Er-147

MAT 6780

$(n, n') \alpha$

68-Er-147

Radionuclide Production Cross Section



64

Incident Energy (MeV)

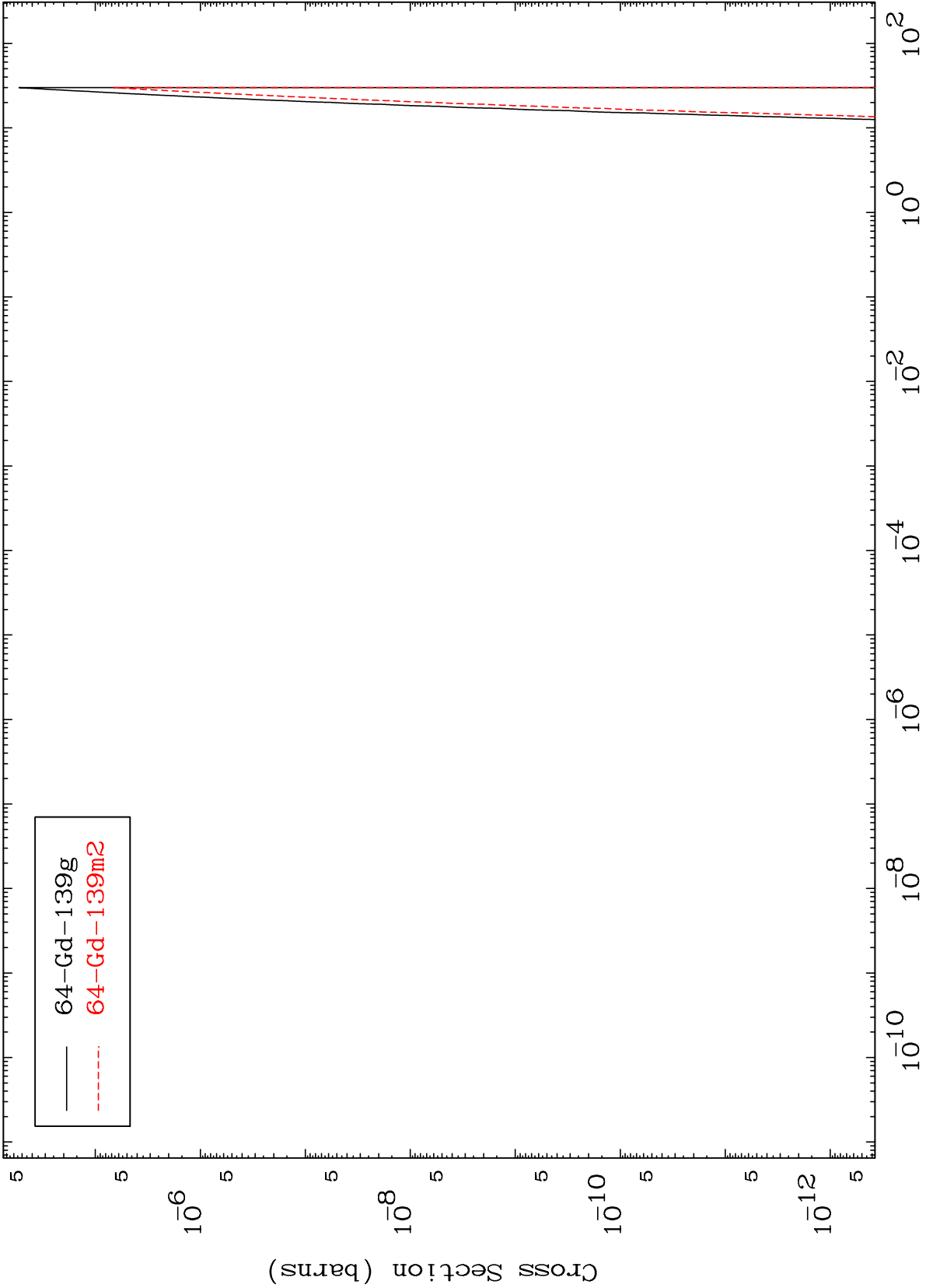
68-Er-147

MAT 6780

(n,n') 2 α

68-Er-147

Radionuclide Production Cross Section



64-Gd-139g
64-Gd-139m2

65

Incident Energy (MeV)

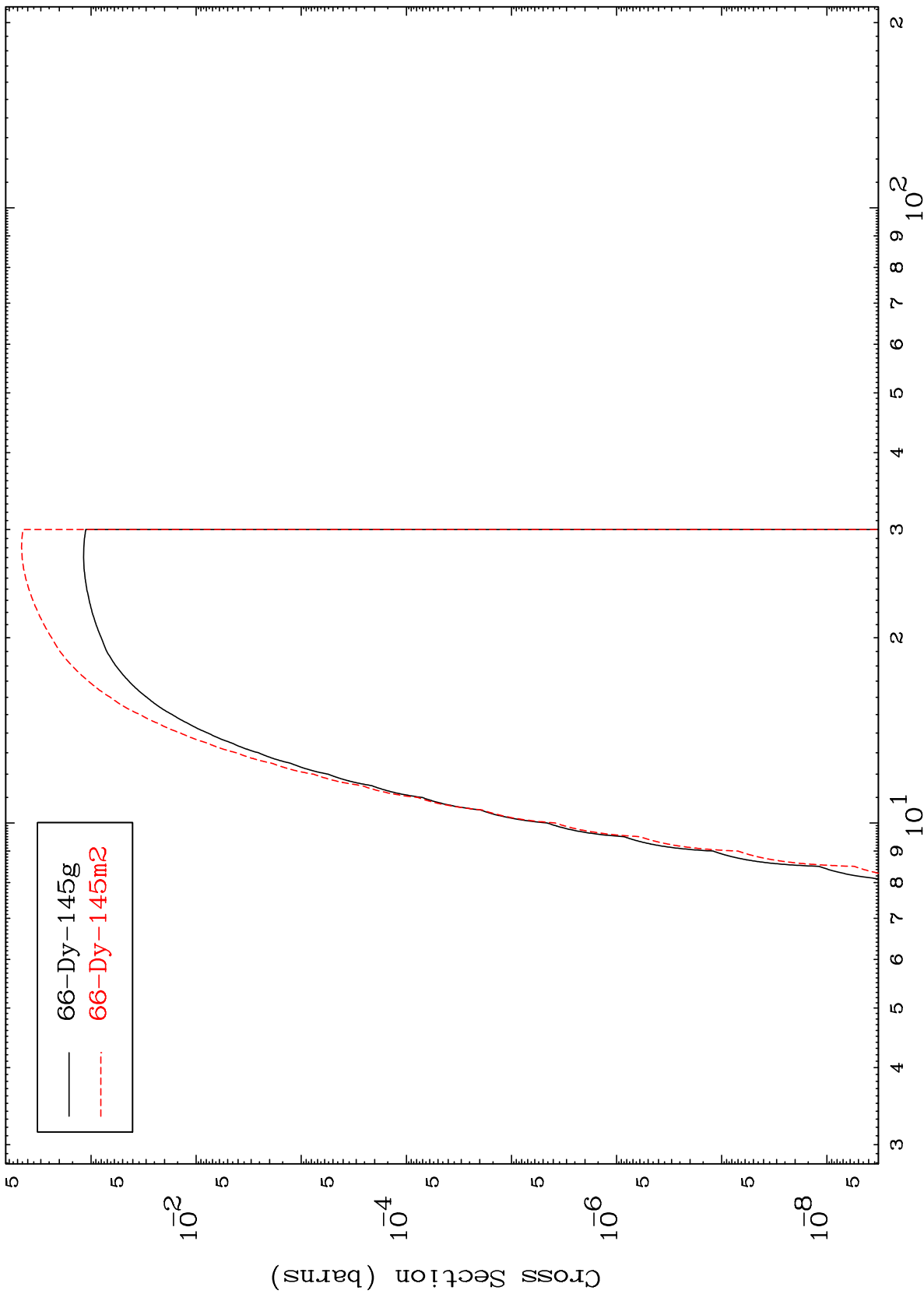
68-Er-147

MAT 6780

(n,2n) p

68-Er-147

Radionuclide Production Cross Section



Incident Energy (MeV)

68-Er-147

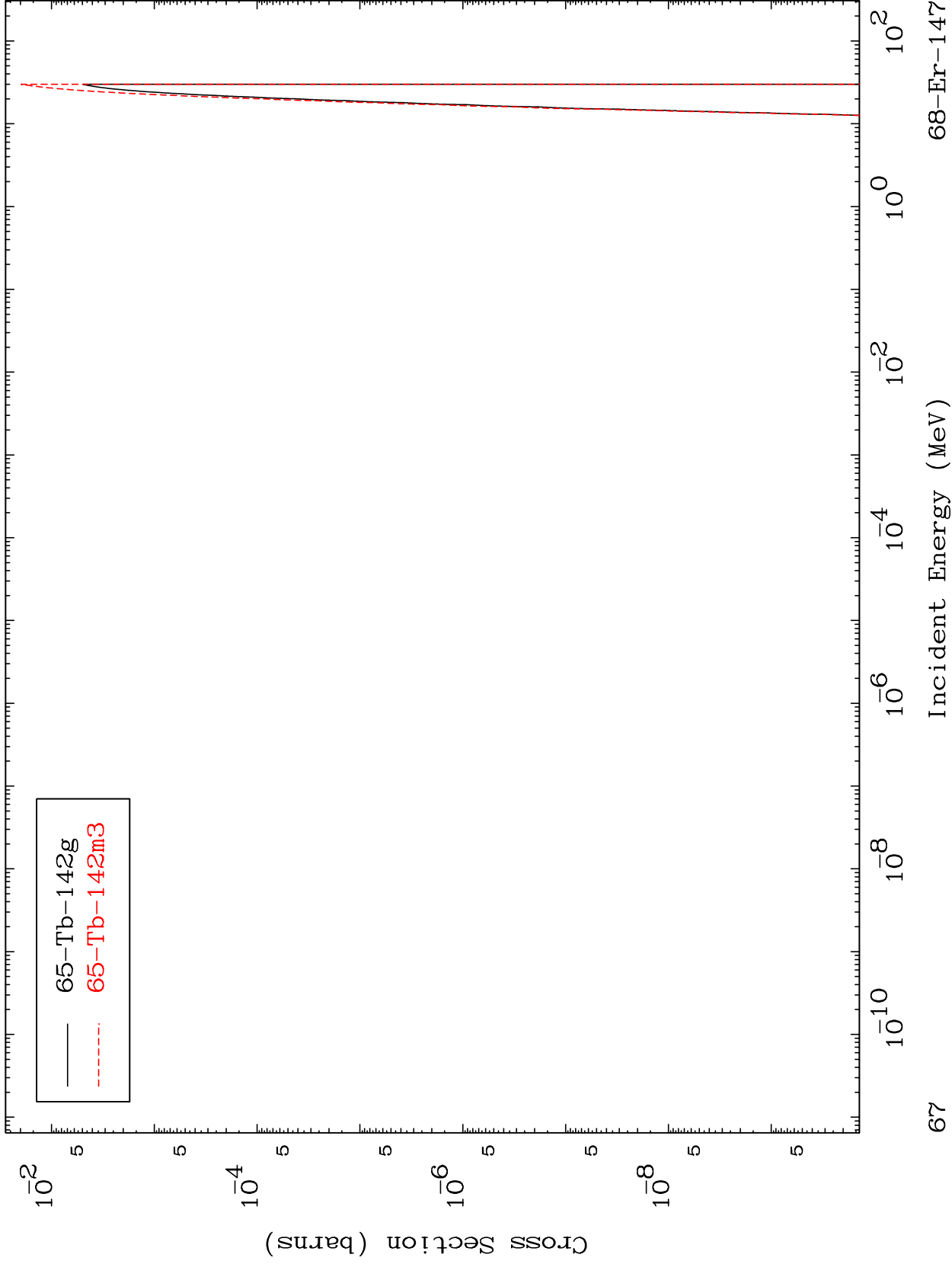
66

MAT 6780

(n,n') p α

68-Er-147

Radionuclide Production Cross Section



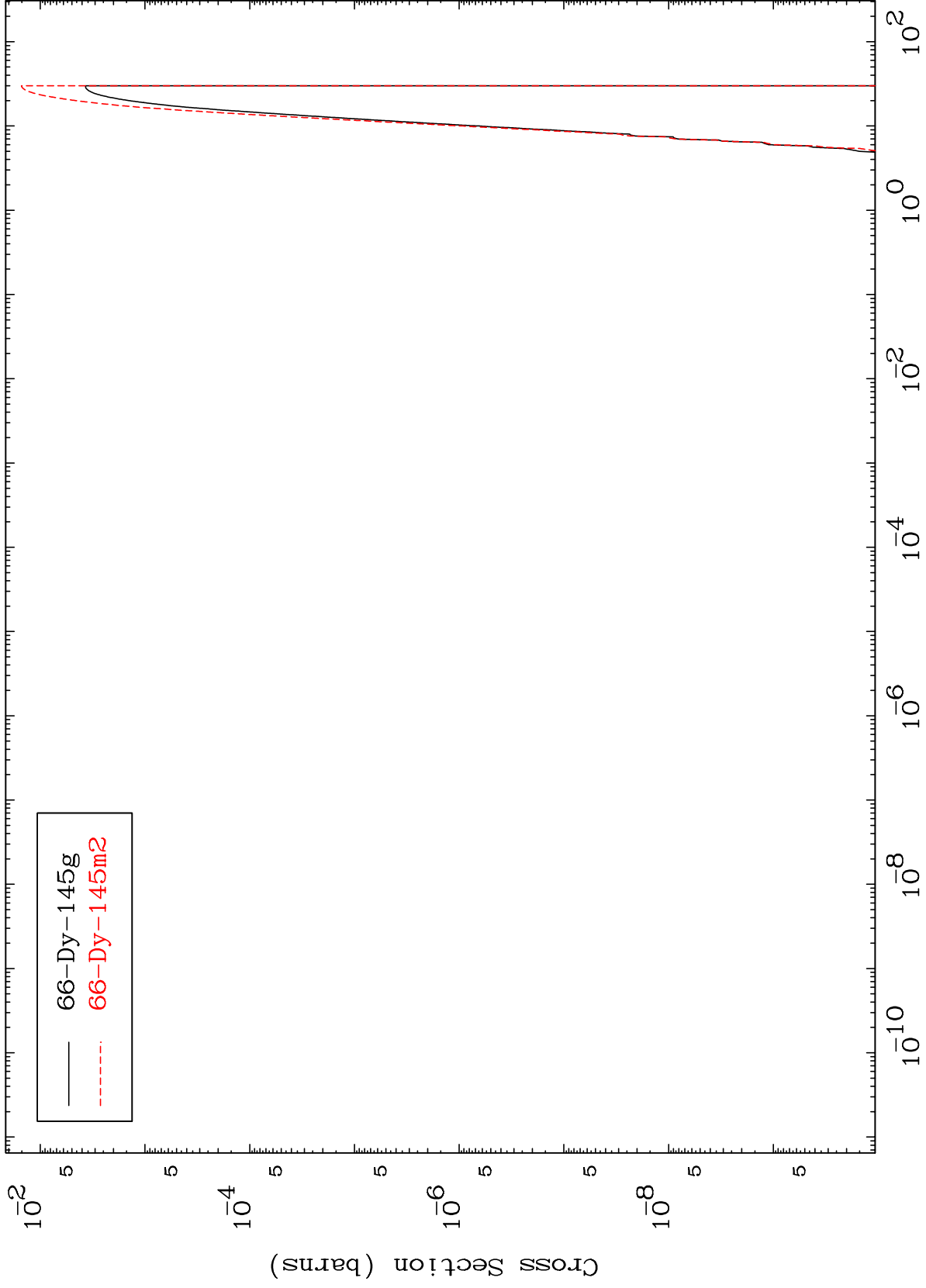
67

MAT 6780

(n,He-3)

68-Er-147

Radionuclide Production Cross Section



68

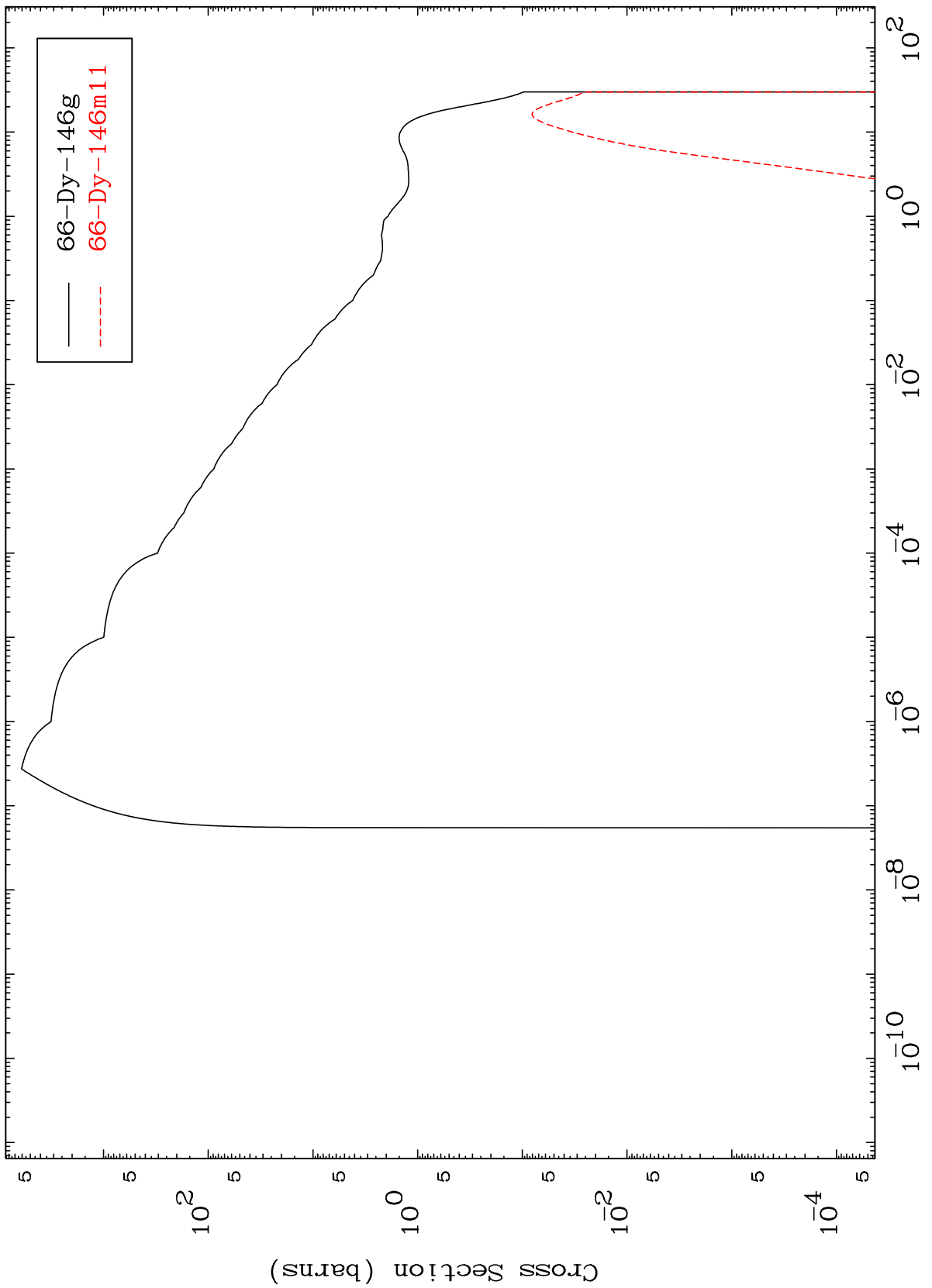
Incident Energy (MeV)

68-Er-147

MAT 6780

68-Er-147

(n,2p)
Radionuclide Production Cross Section



69

68-Er-147

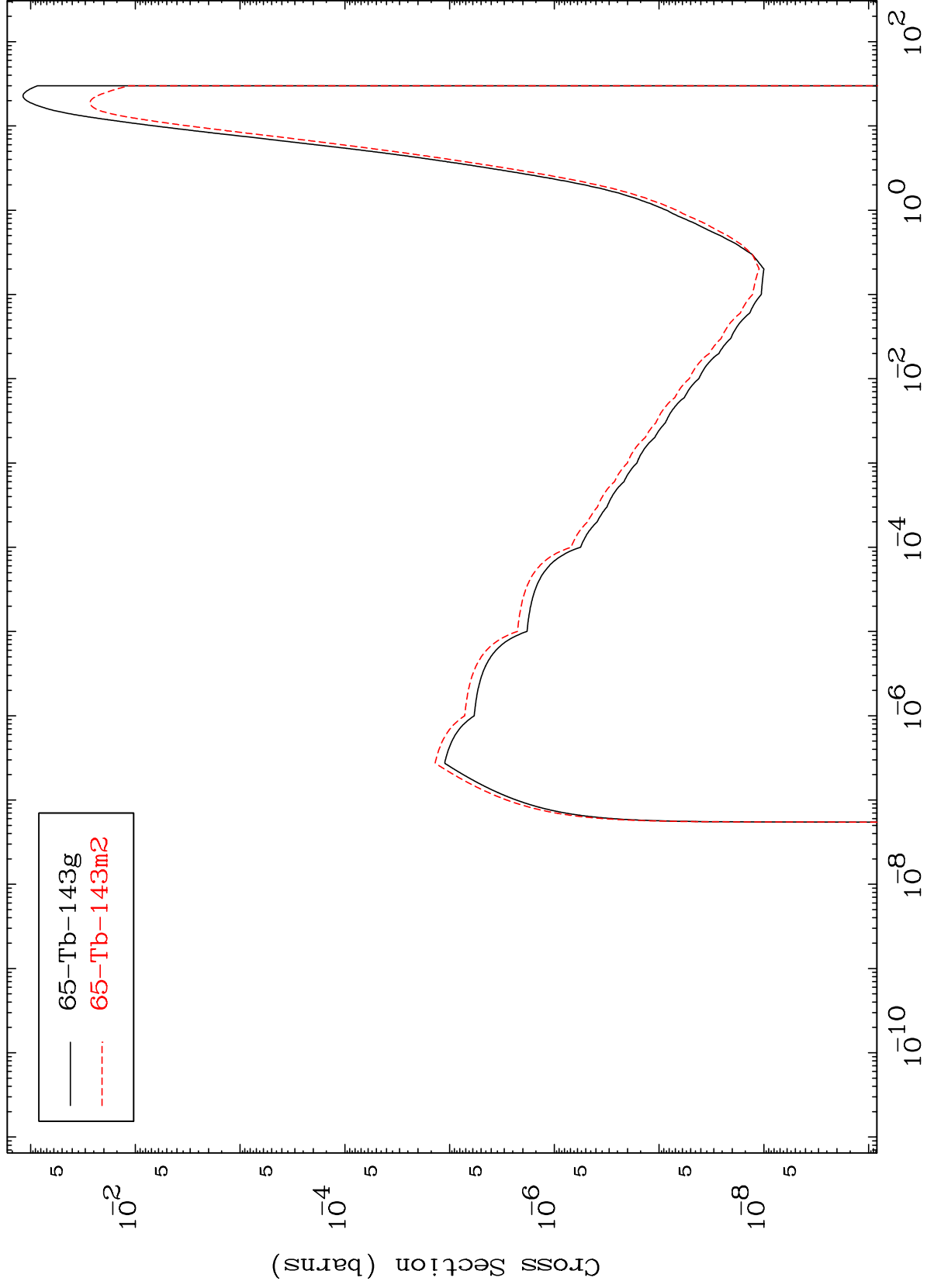
Incident Energy (MeV)

MAT 6780

(n,p) α

68-Er-147

Radionuclide Production Cross Section



70

Incident Energy (MeV)

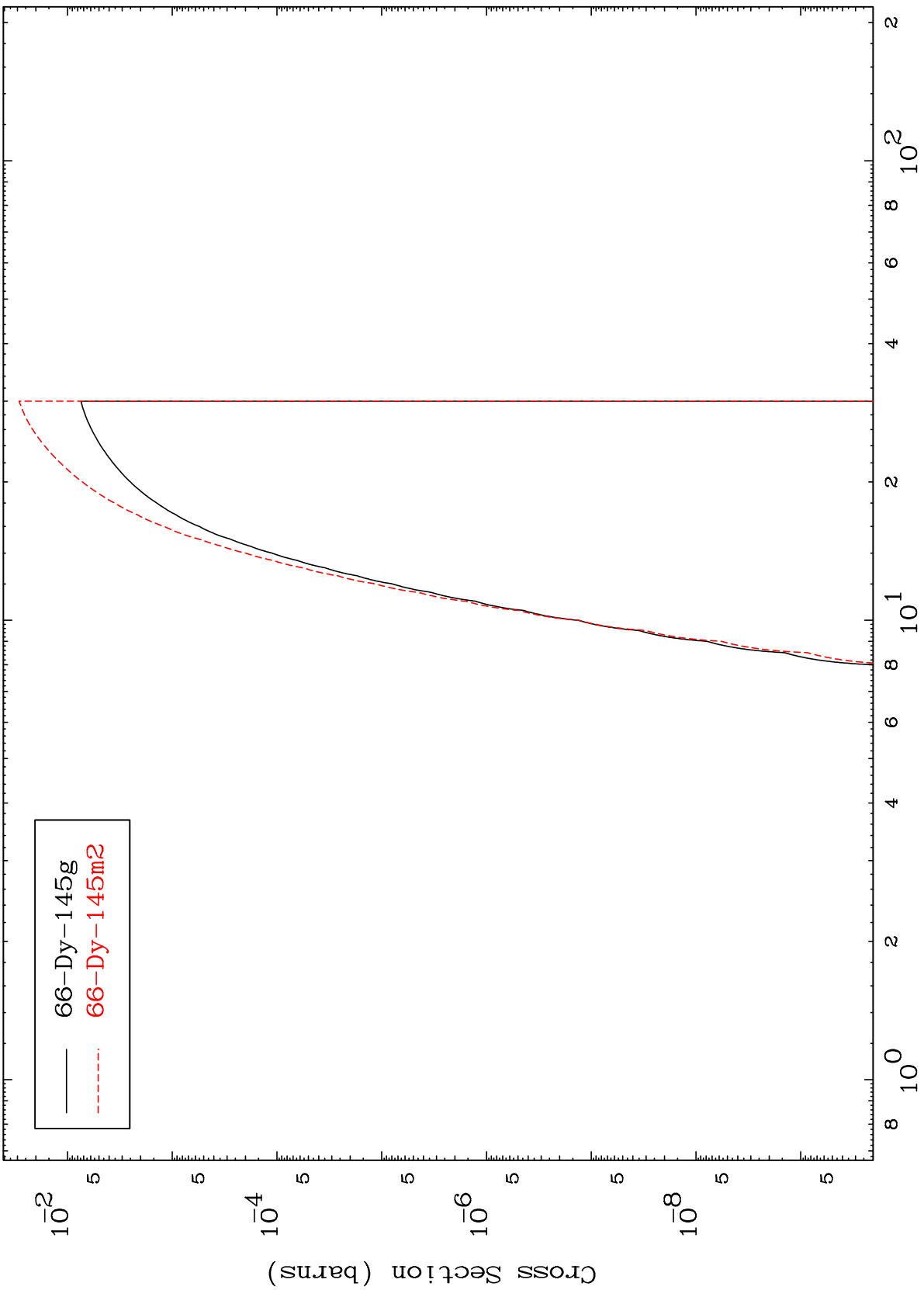
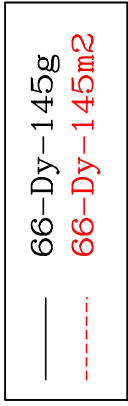
68-Er-147

MAT 6780

(n,p) d

68-Er-147

Radionuclide Production Cross Section



71

Incident Energy (MeV)

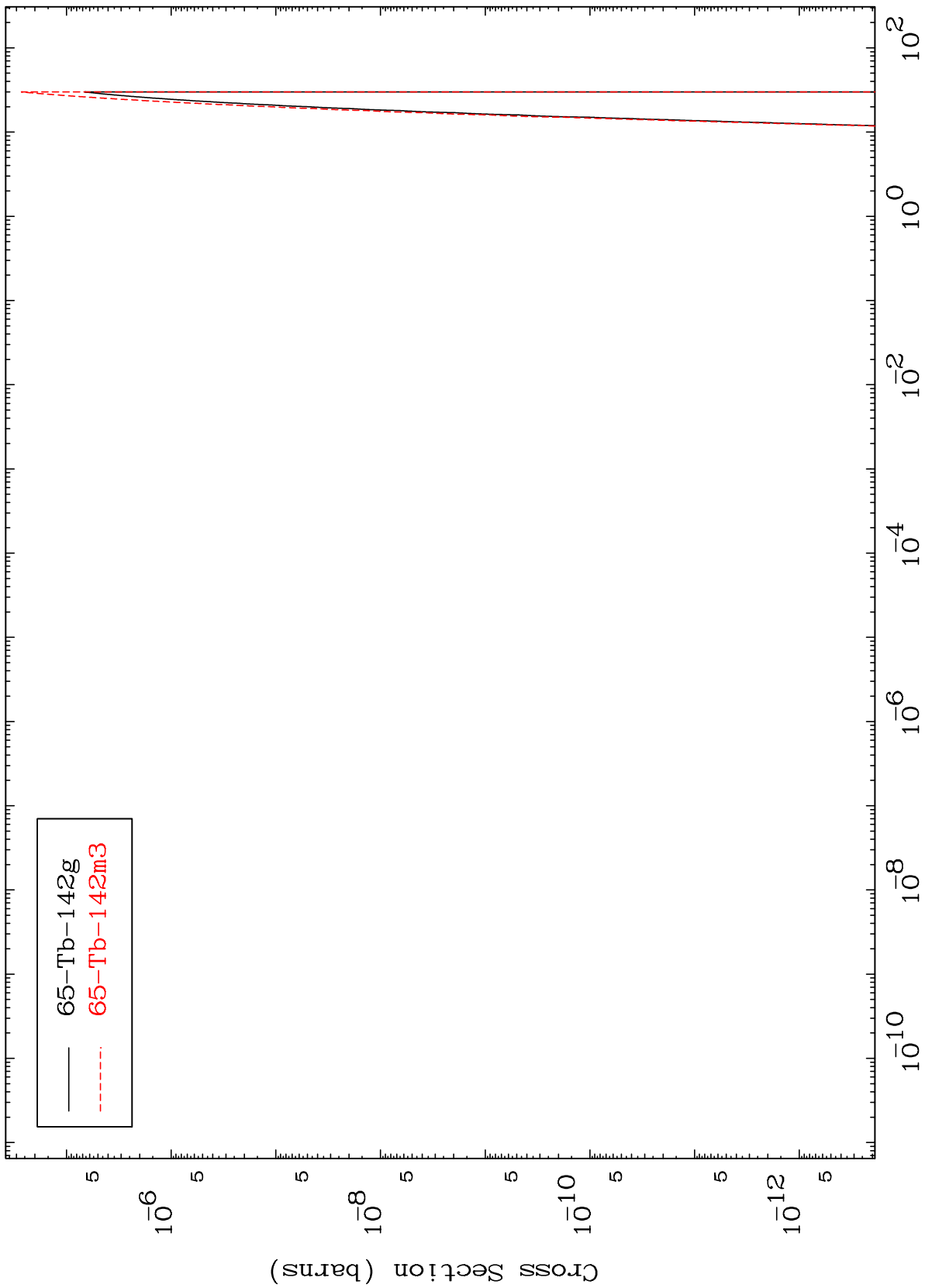
68-Er-147

MAT 6780

(n,d) α

68-Er-147

Radionuclide Production Cross Section



72

Incident Energy (MeV)

68-Er-147