

Program EVALPLOT
(Version 2021-1)

by

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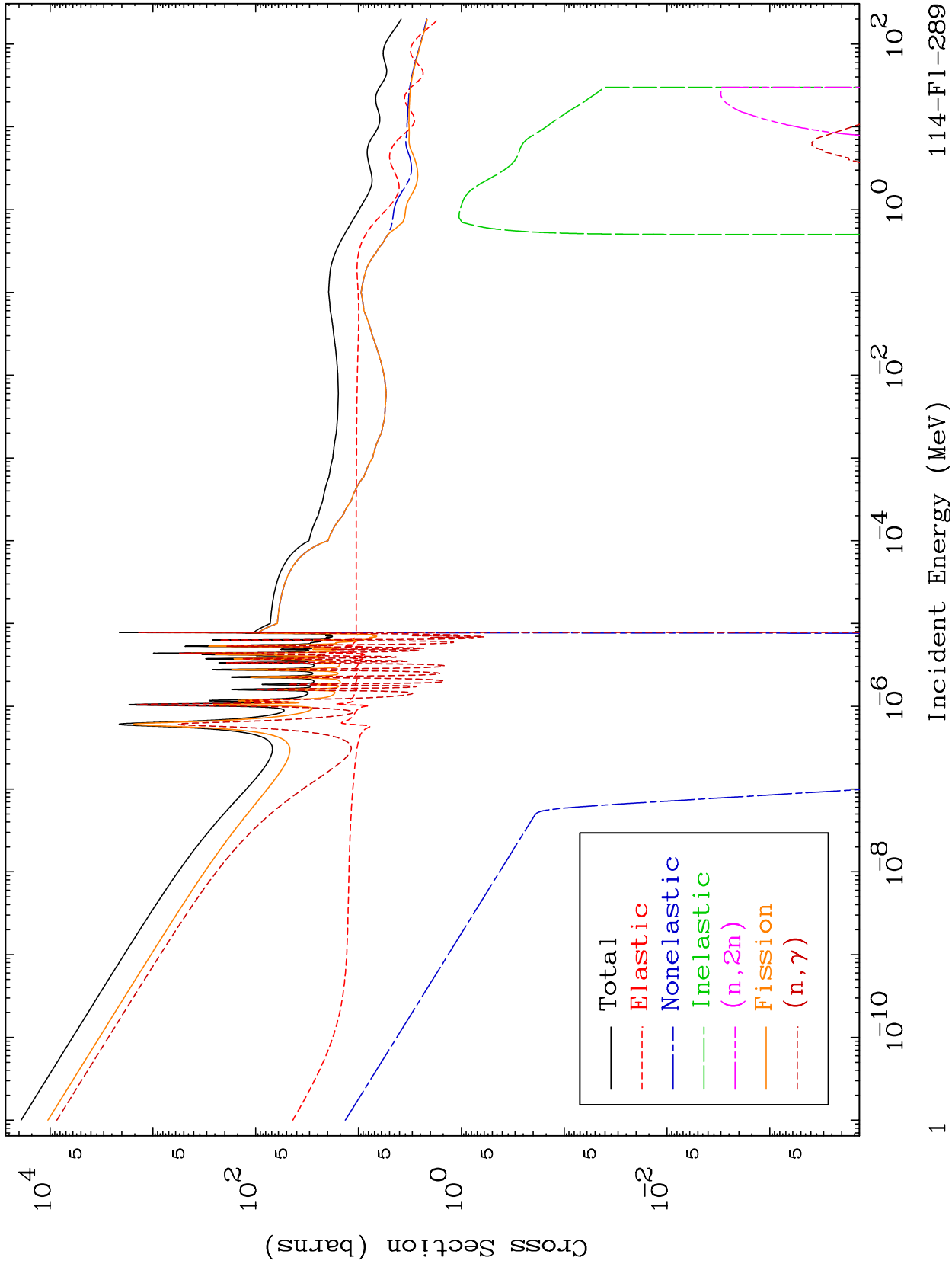
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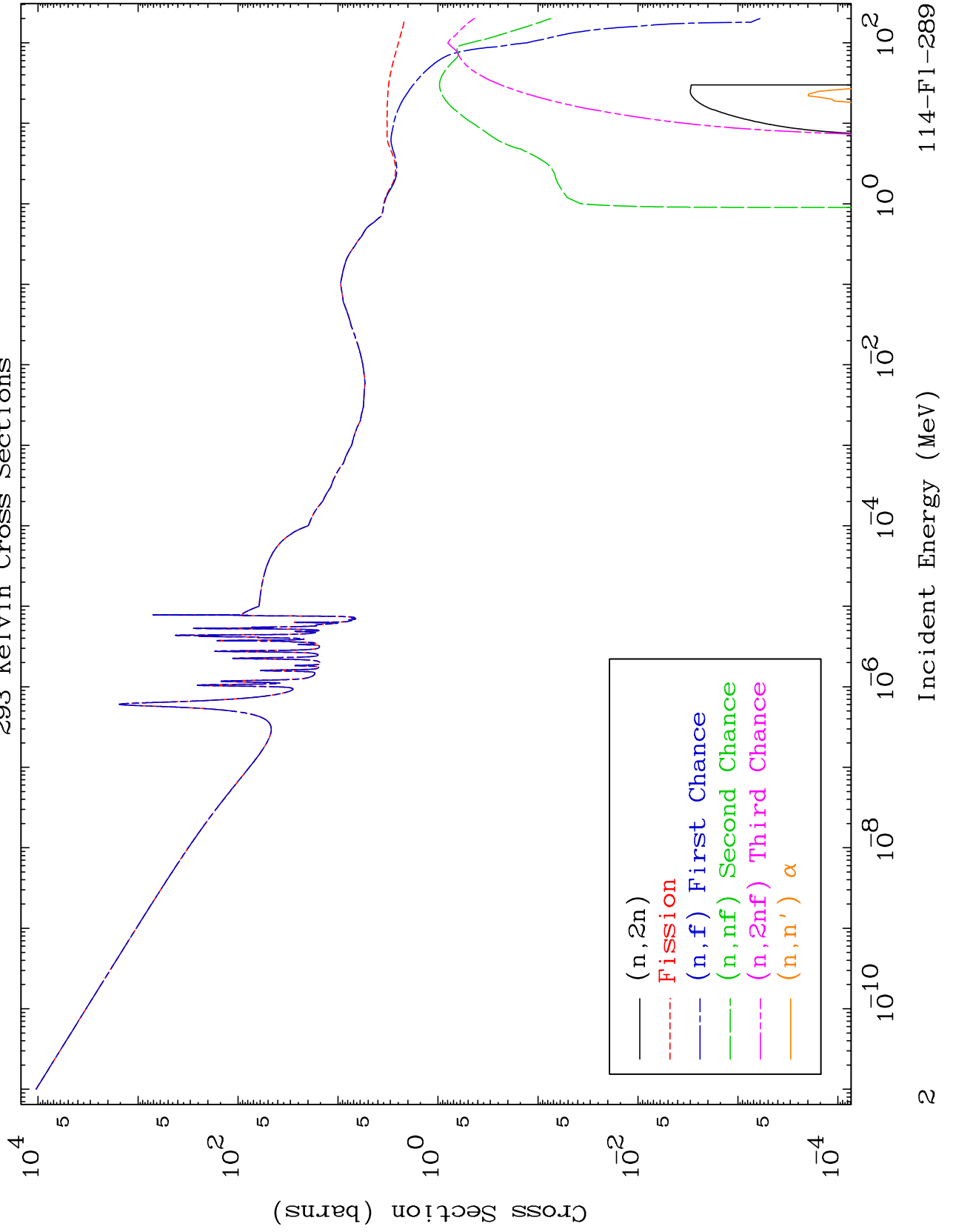
Press Mouse Button to Start

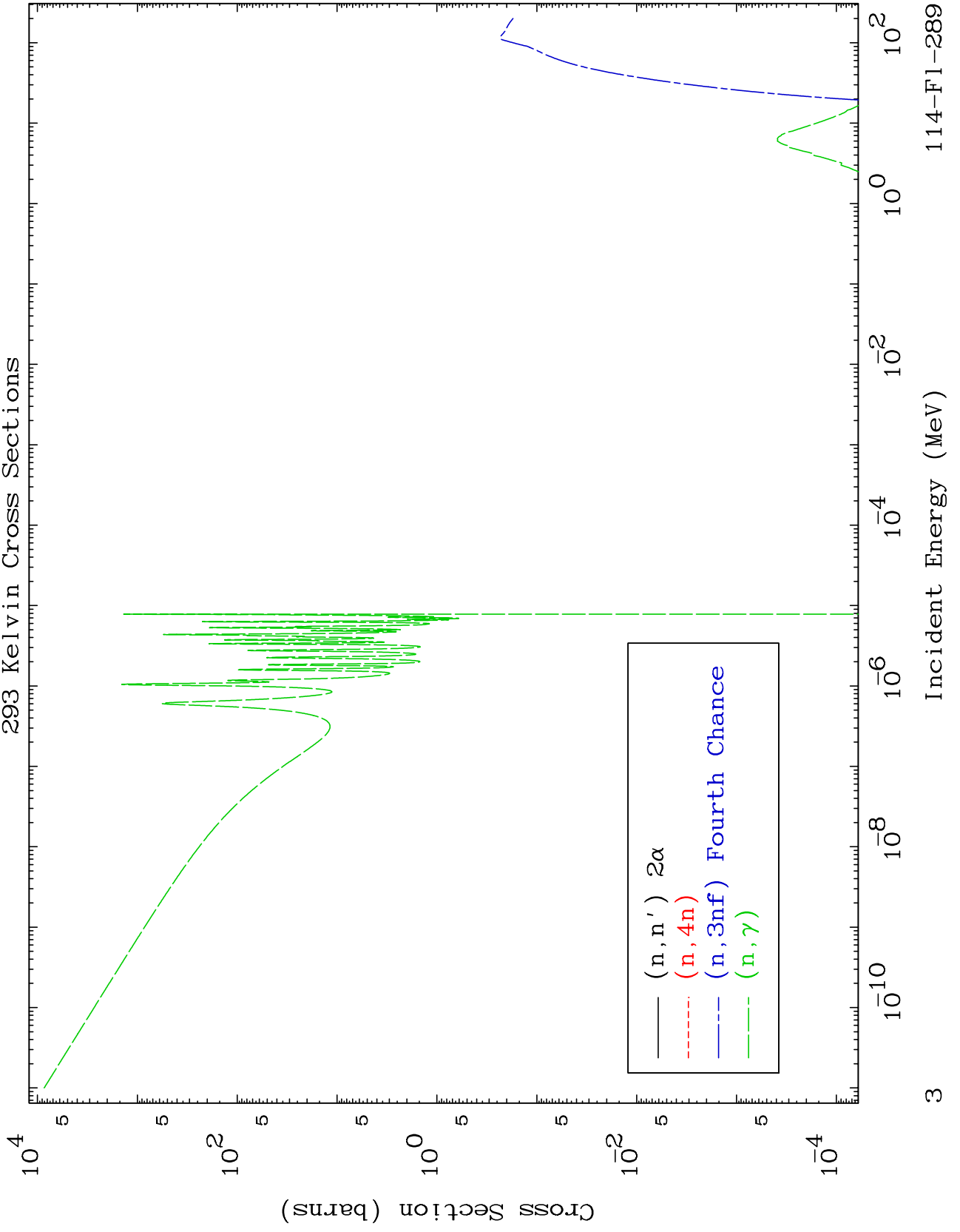
MAT 1489

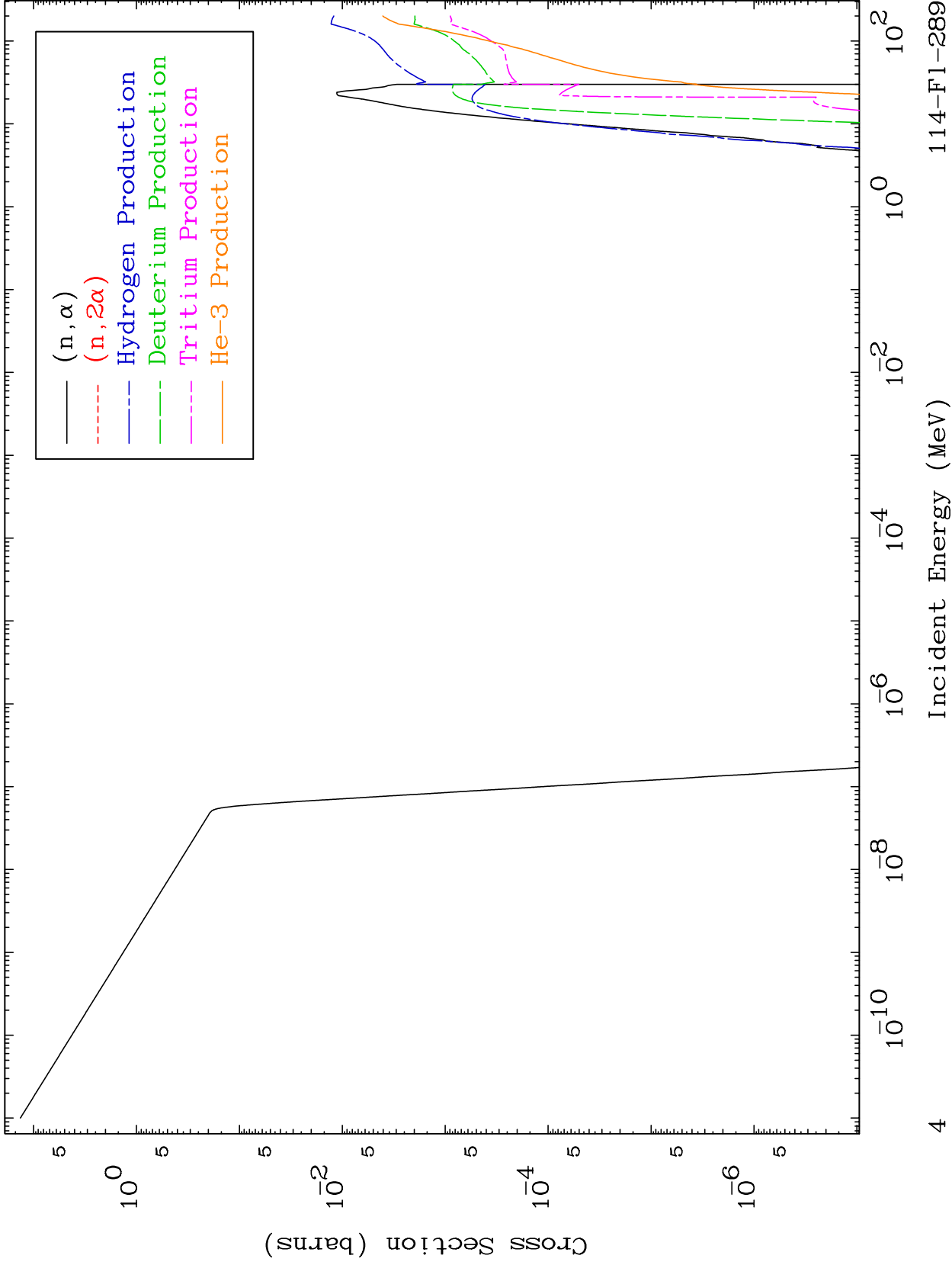
Neutron Major
293 Kelvin Cross Sections

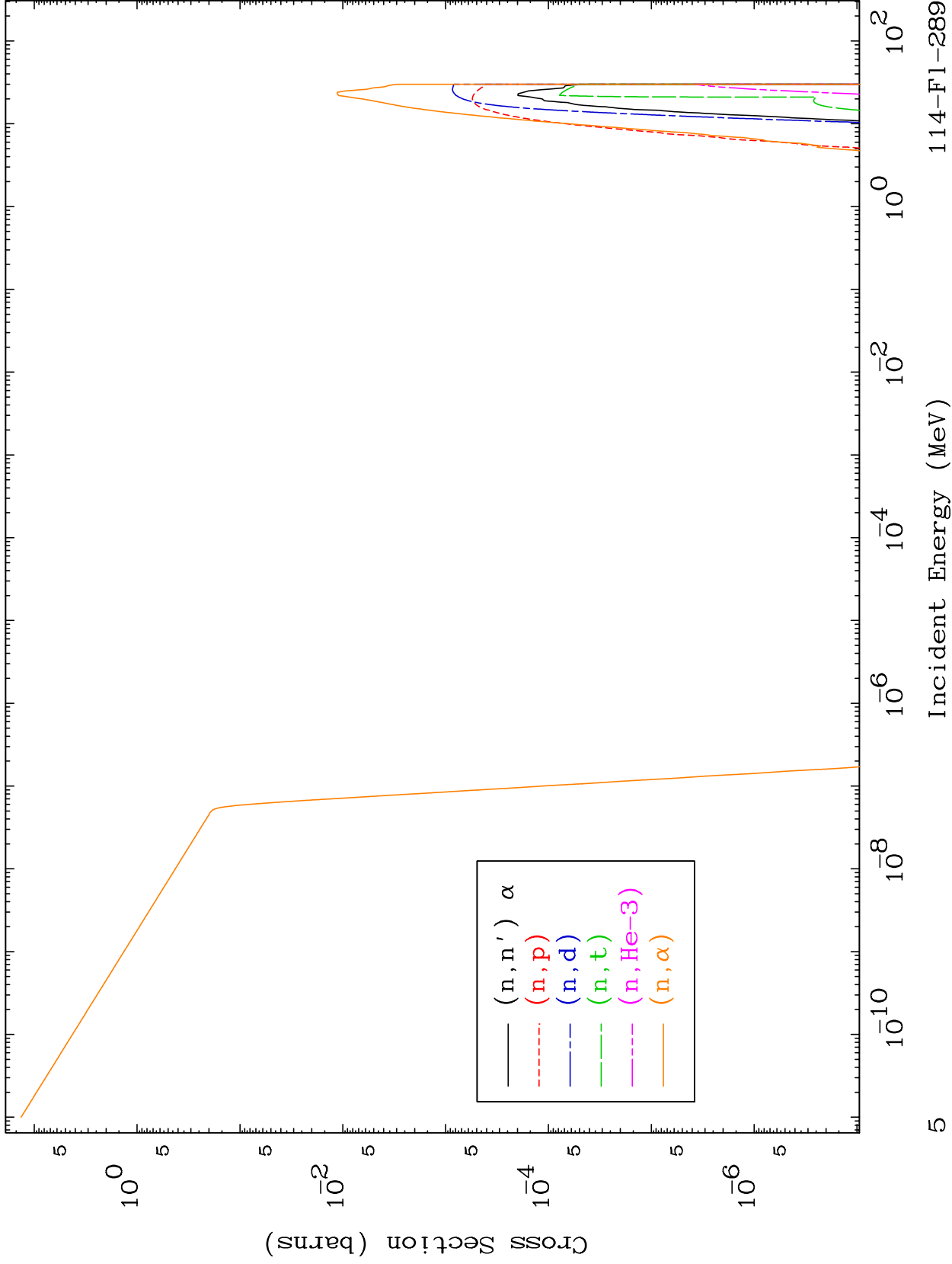
114-F1-289

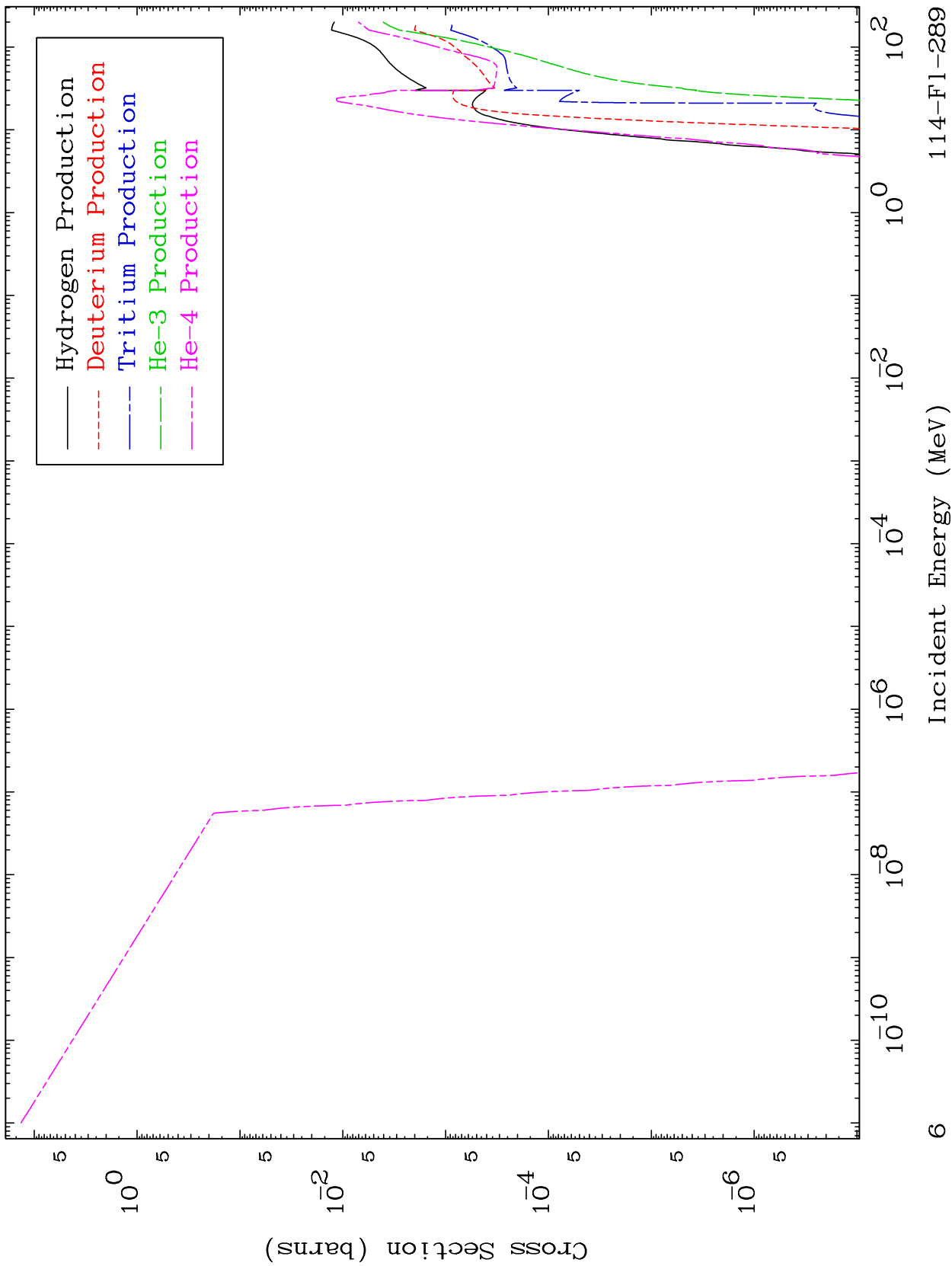








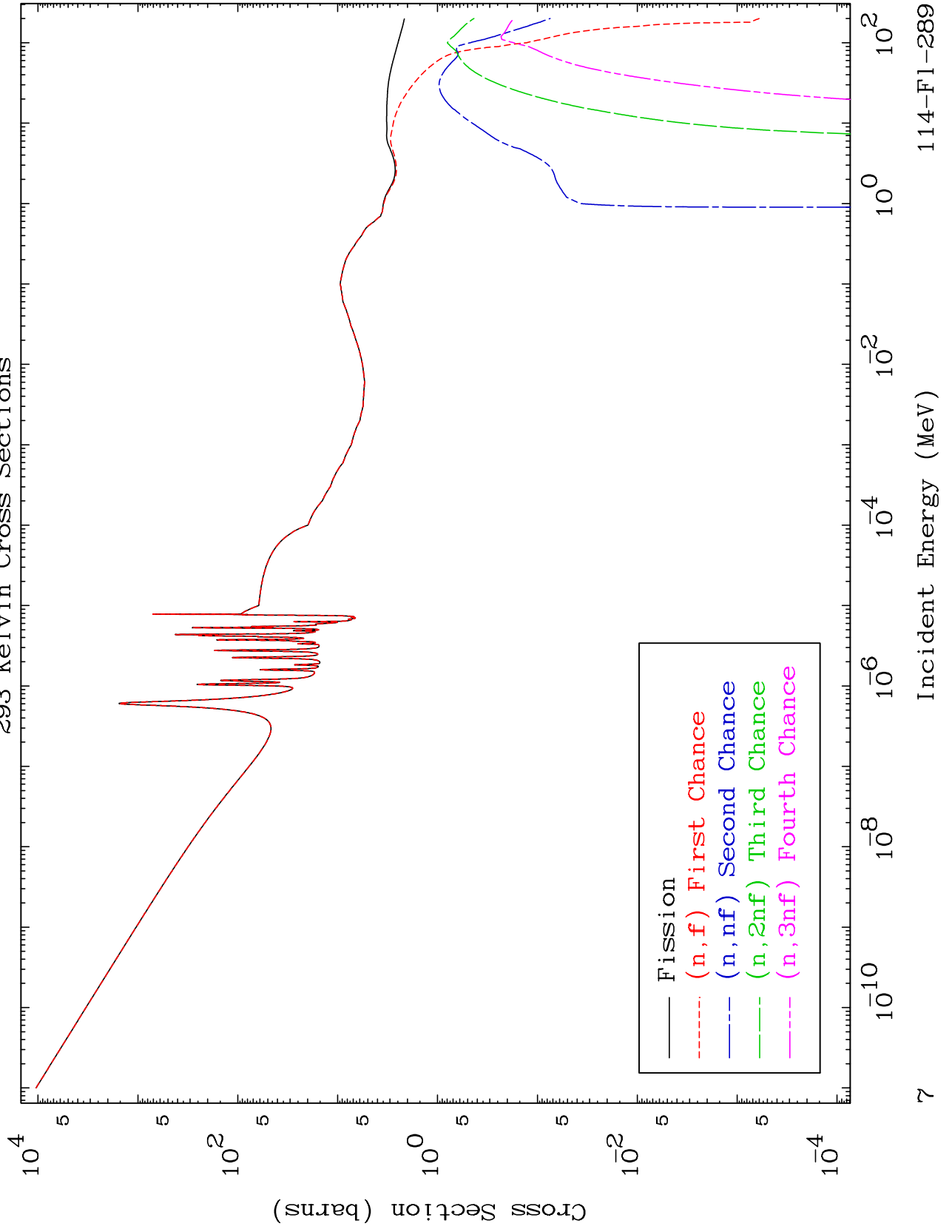




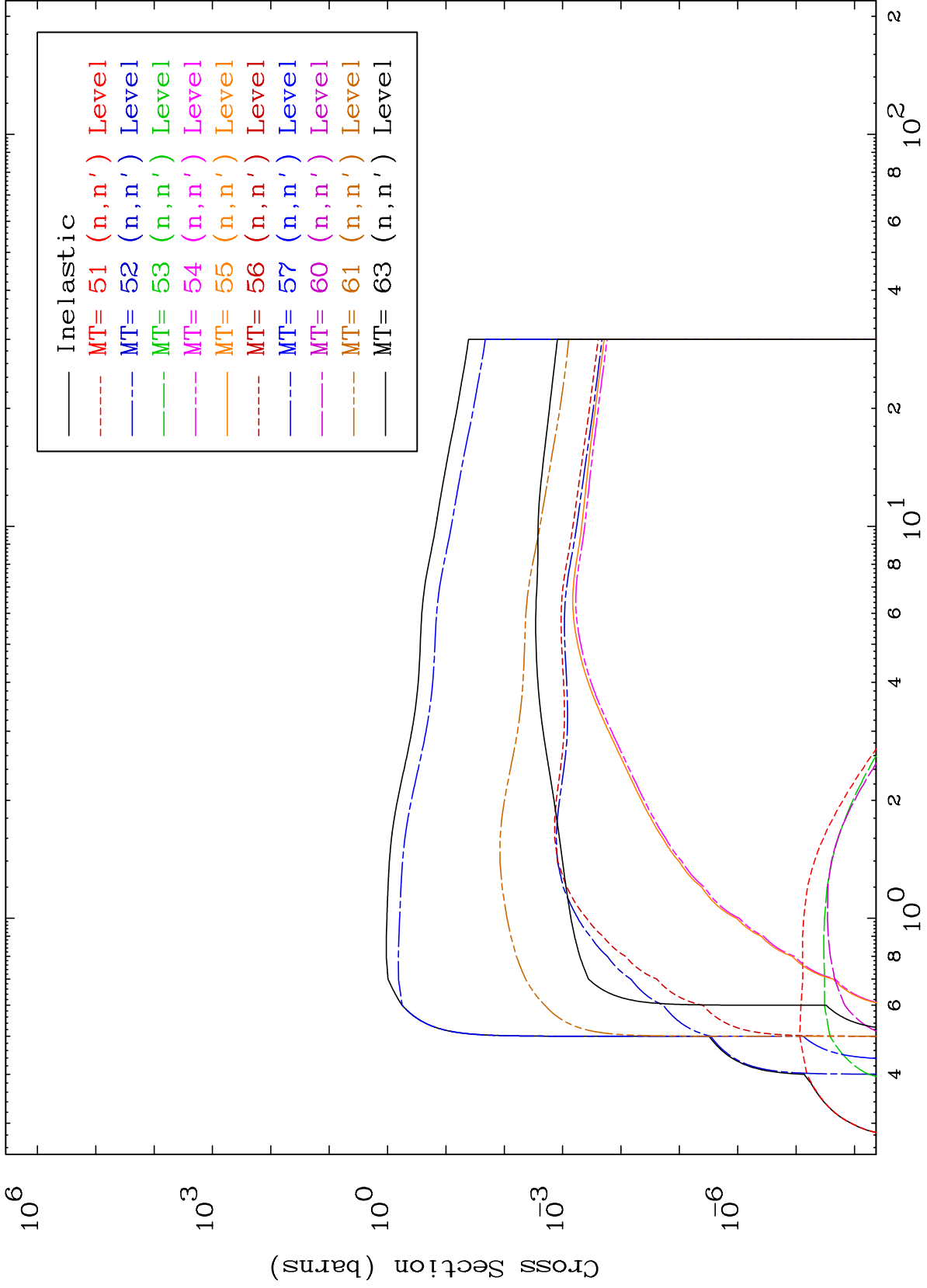
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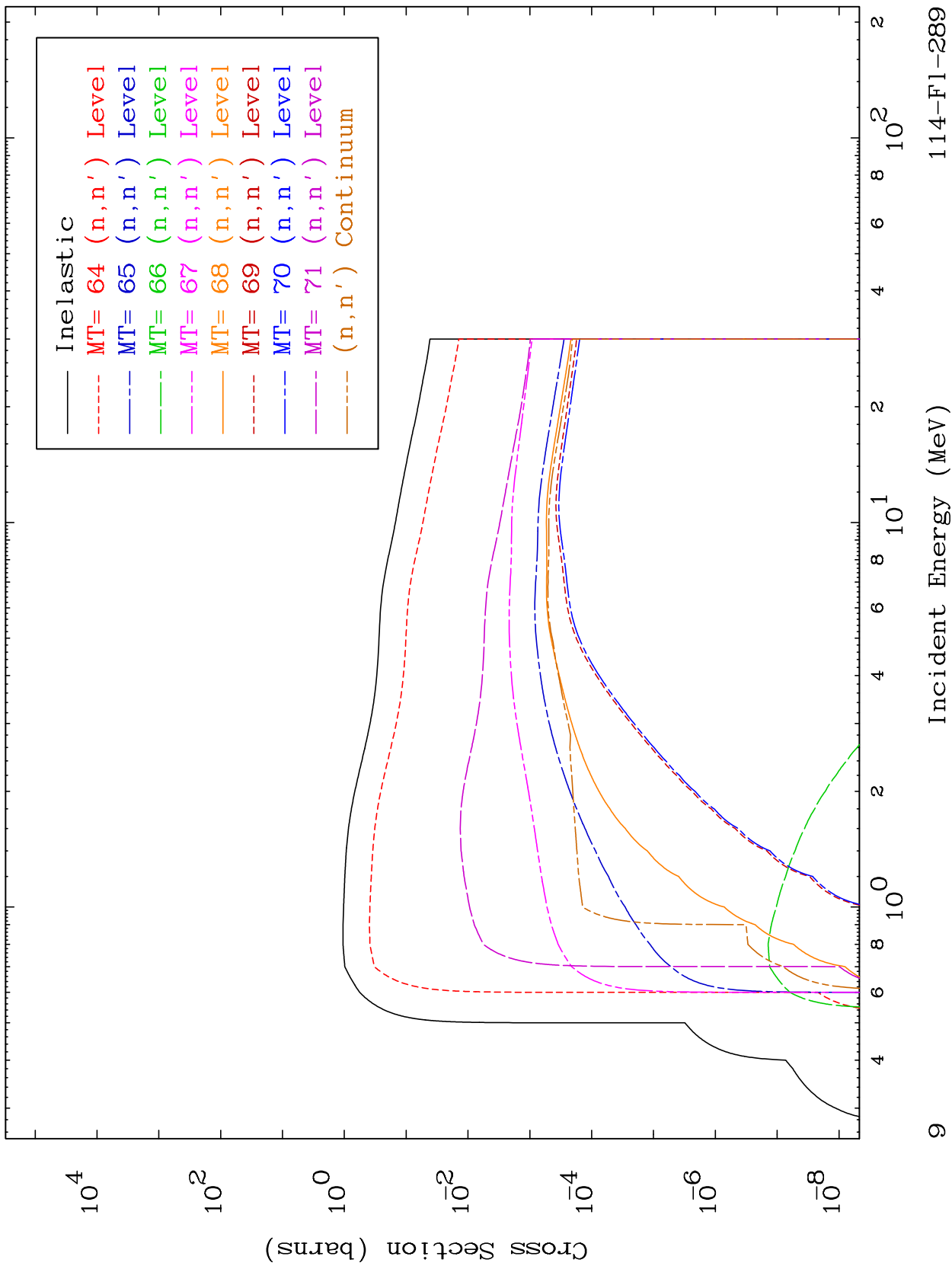
Fission
293 Kelvin Cross Sections

114-FI-289



293 Kelvin Cross Sections

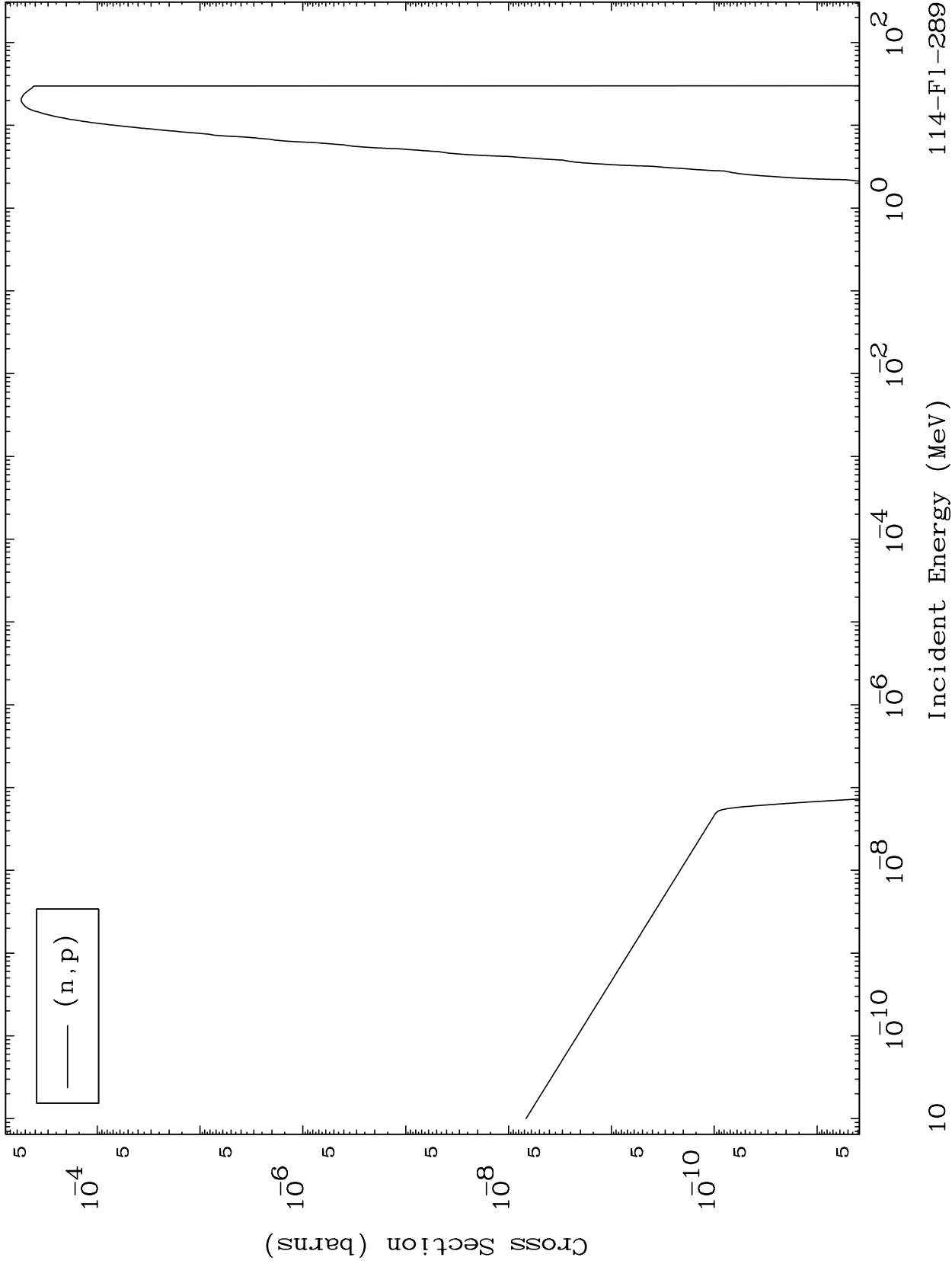




MAT 1489

(n,p) Levels
293 Kelvin Cross Sections

114-F1-289

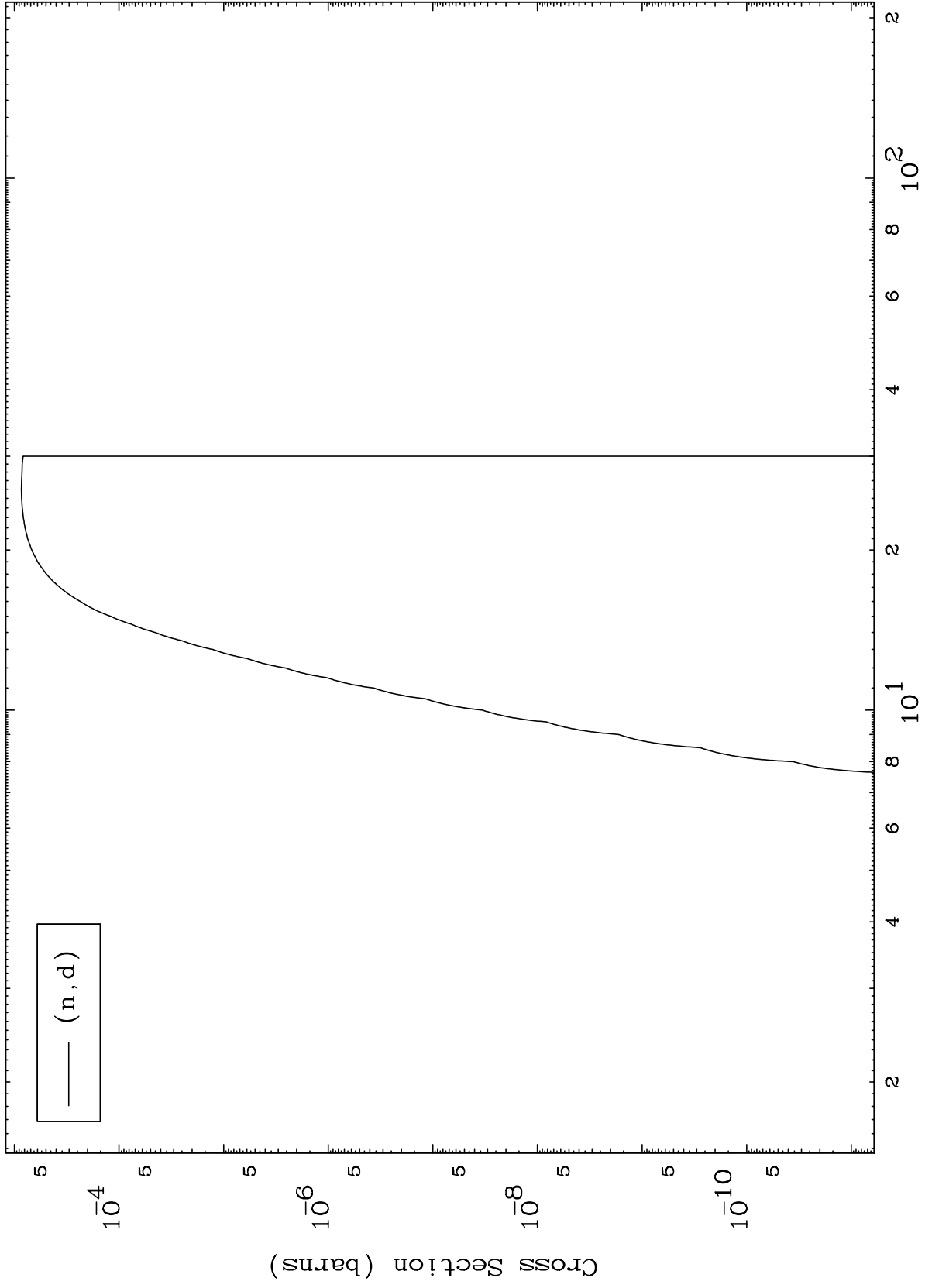


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MAT 1489

(n,d) Levels
293 Kelvin Cross Sections

114-F1-289



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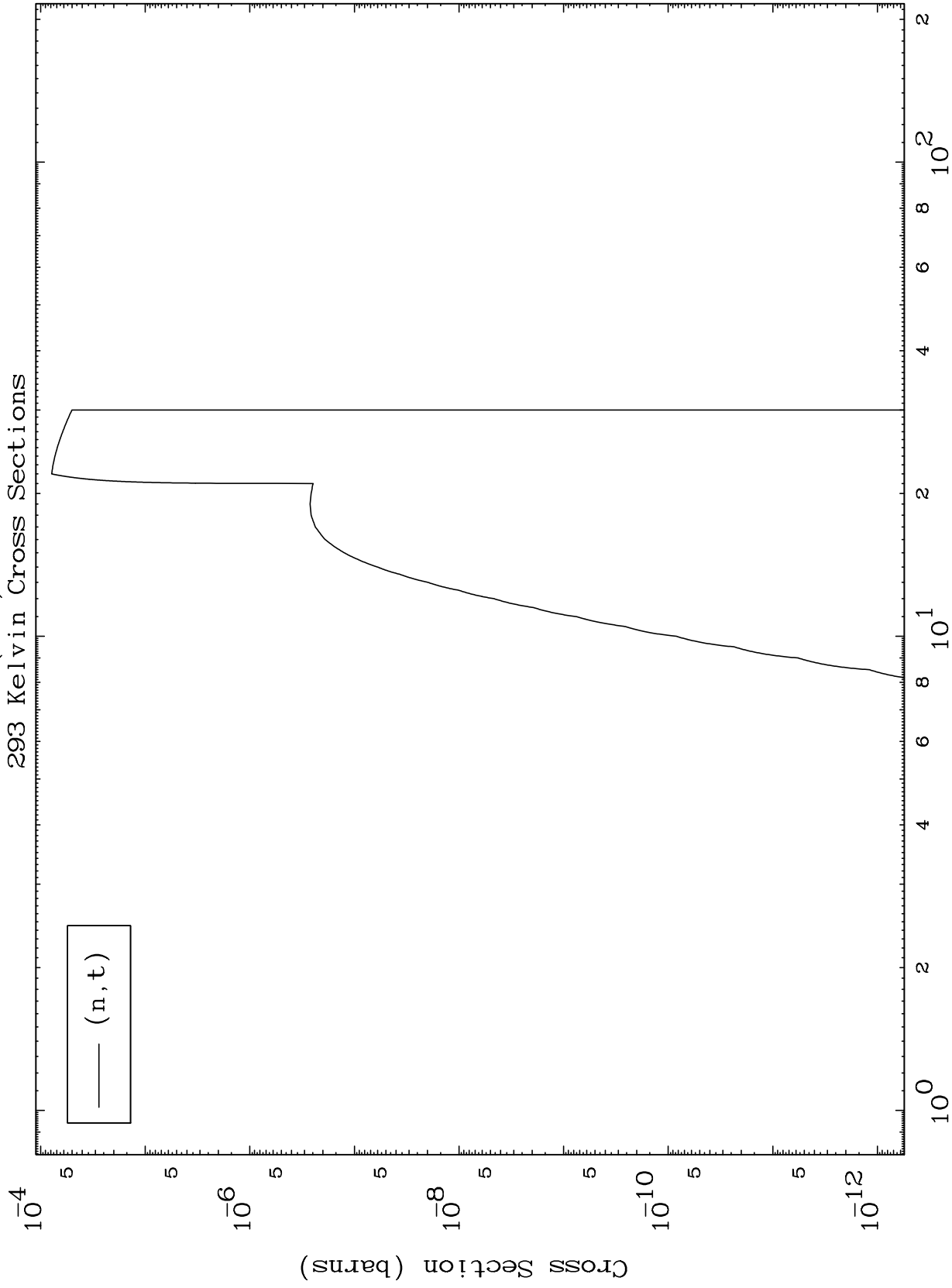
Incident Energy (MeV)

114-F1-289

MAT 1489

(n,t) Levels
293 Kelvin Cross Sections

114-F1-289



12

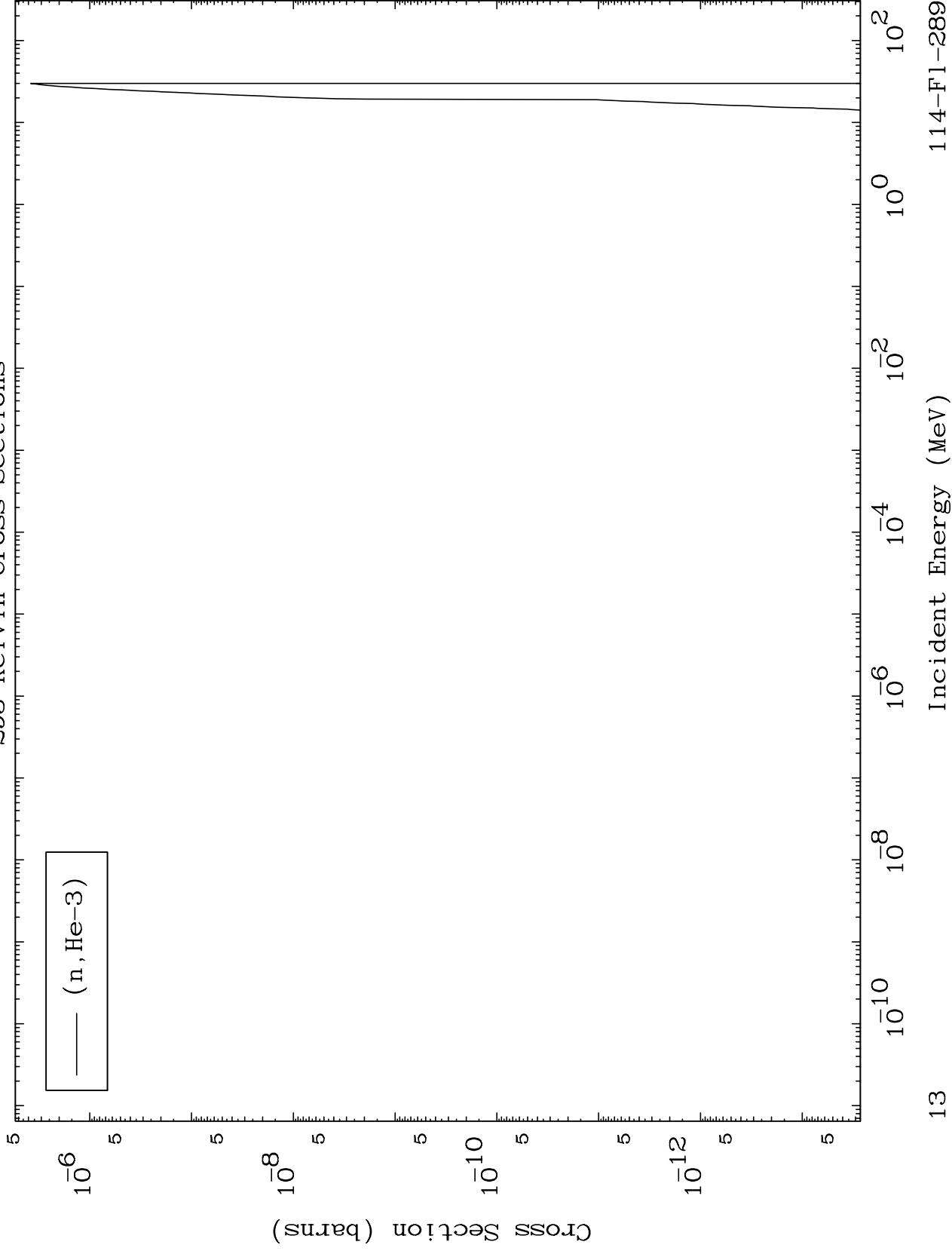
Incident Energy (MeV)

114-F1-289

MAT 1489

(n,He3) Levels
293 Kelvin Cross Sections

114-F1-289



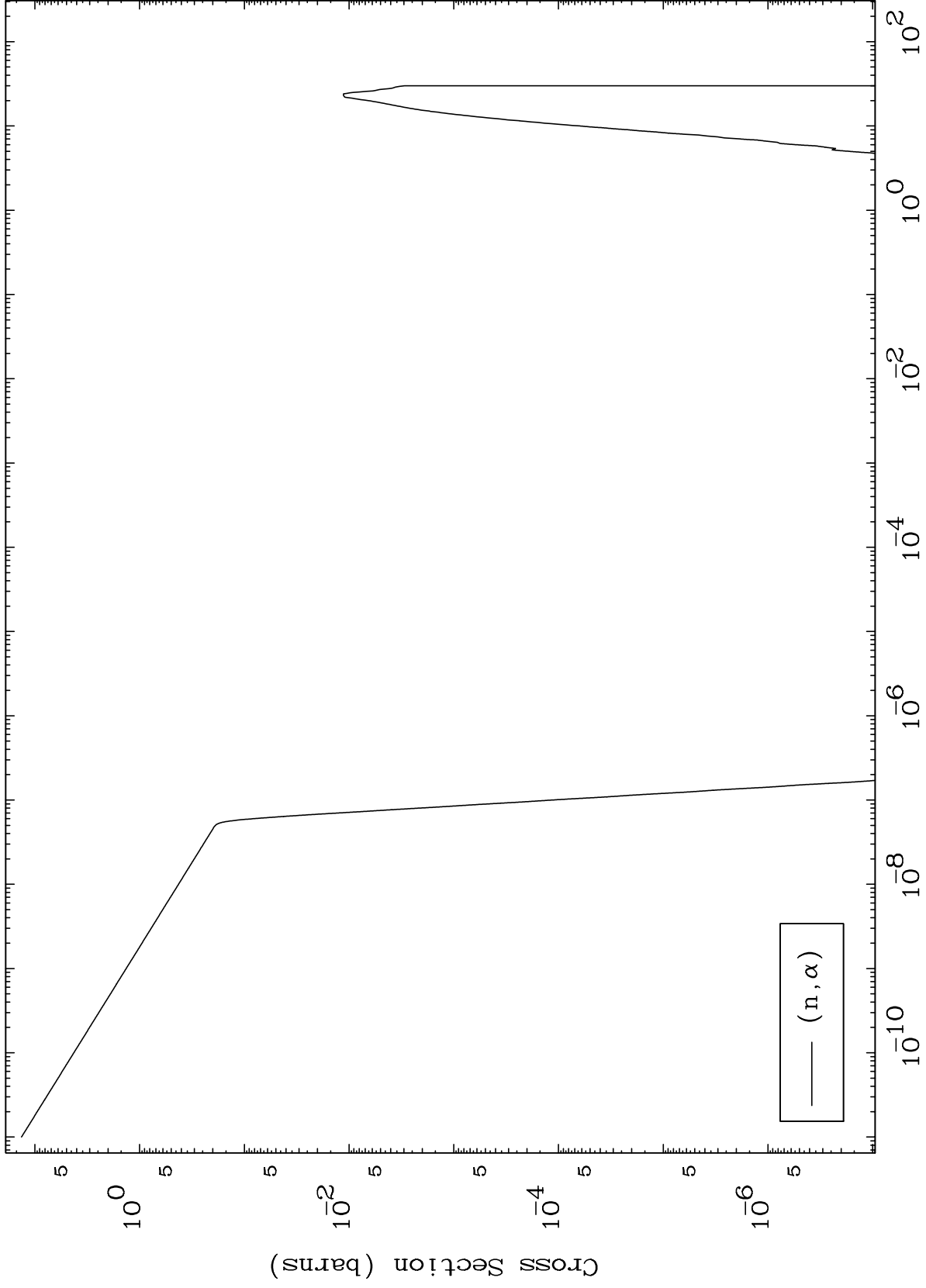
13

114-F1-289

MAT 1489

(n, α) Levels
293 Kelvin Cross Sections

114-F1-289



14

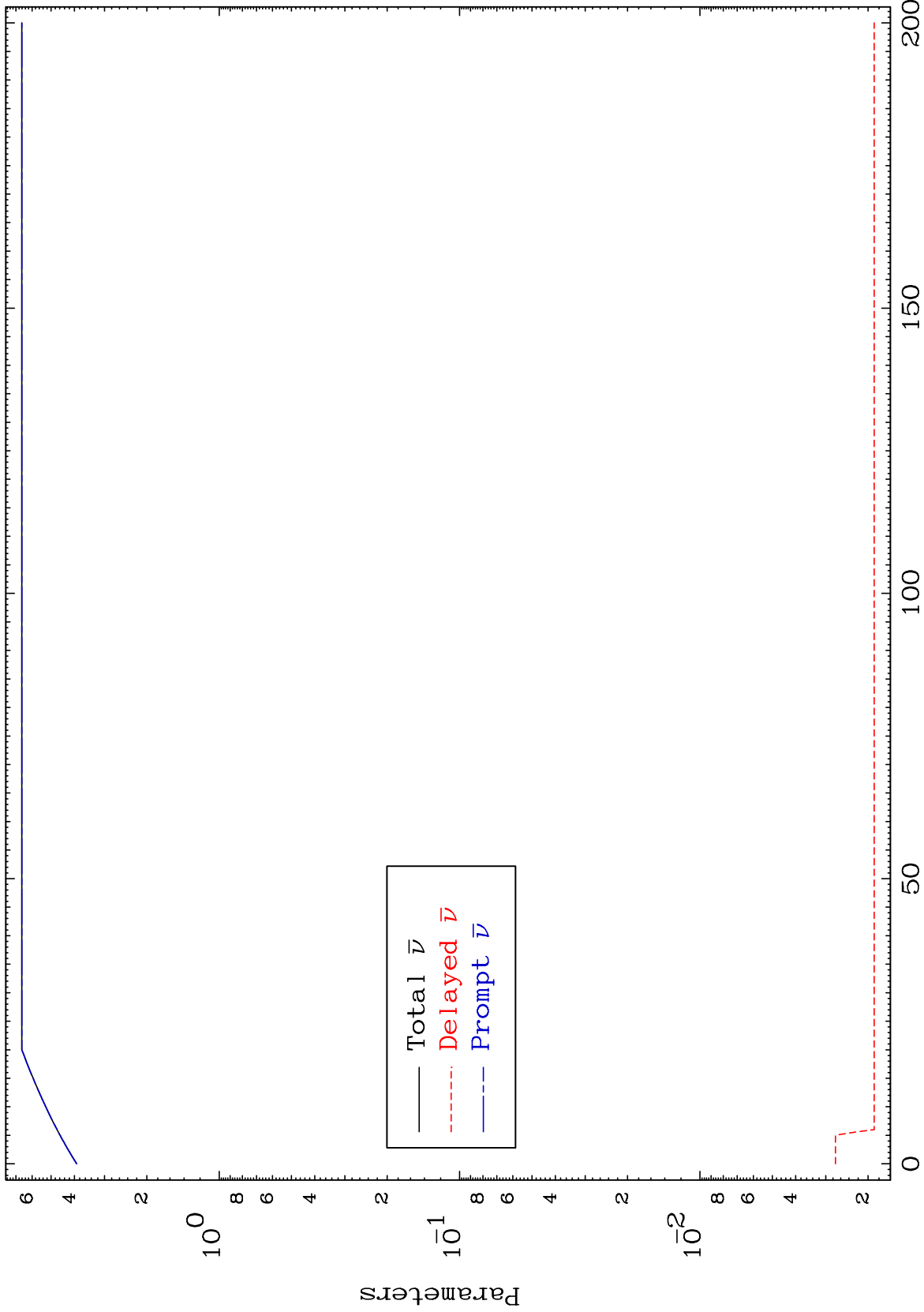
Incident Energy (MeV)

114-F1-289

MAT 1489

Energy Release
Parameters

114-F1-289



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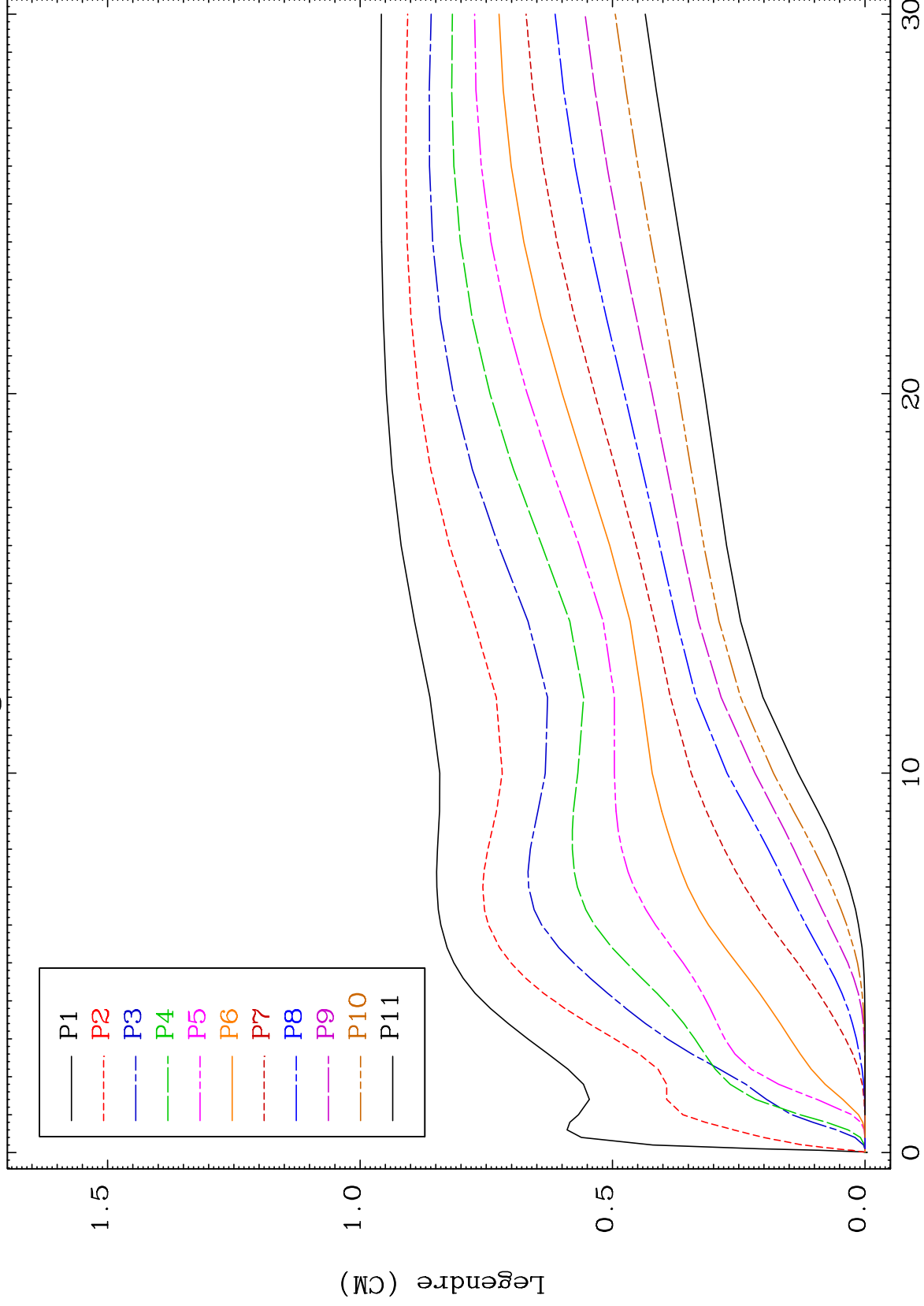
Incident Energy (MeV)

114-F1-289

MAT 1489

Elastic Legendre Coefficients

114-F1-289



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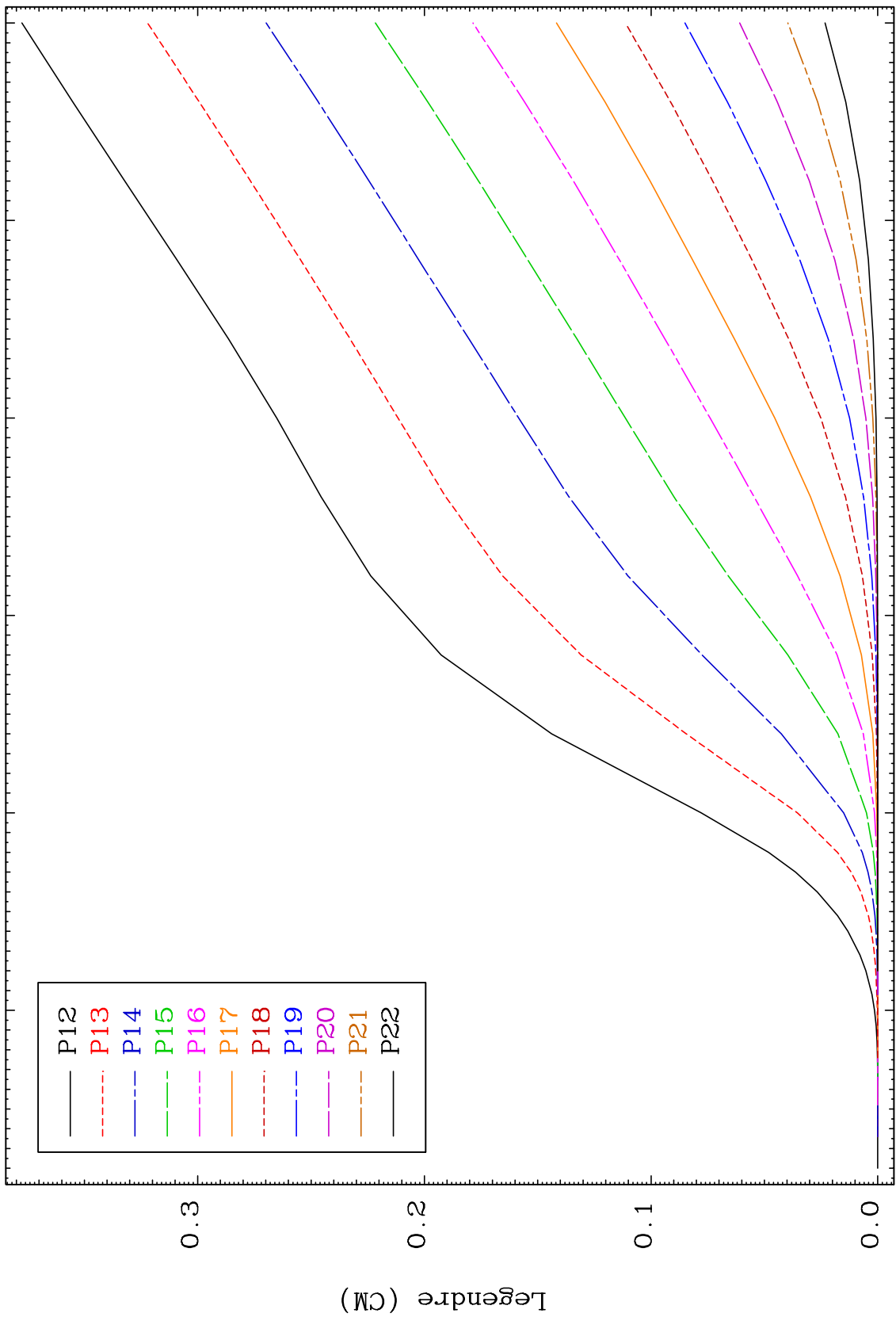
Incident Energy (MeV)

114-F1-289

MAT 1489

Elastic Legendre Coefficients

114-F1-289



17

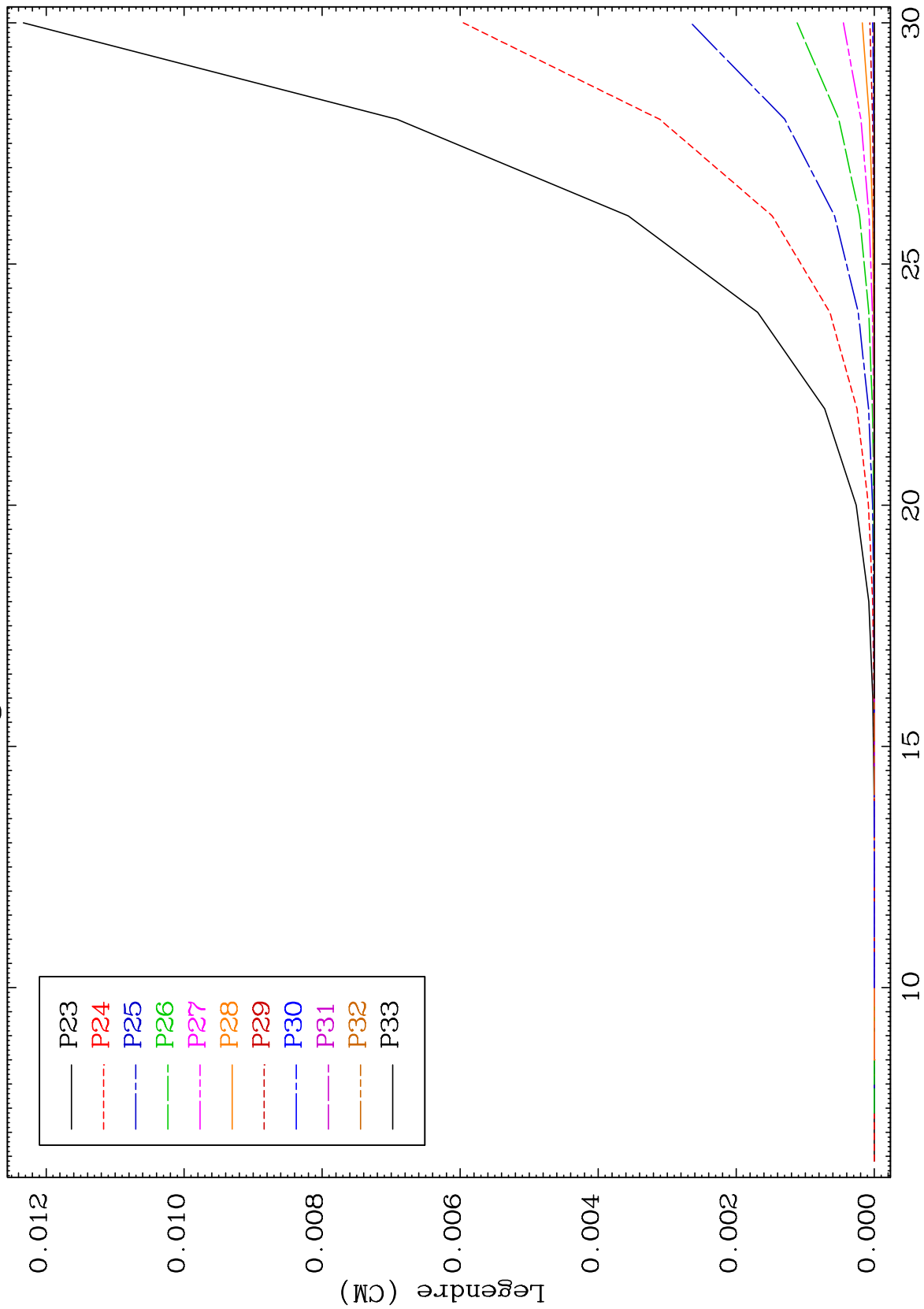
Incident Energy (MeV)

114-F1-289

MAT 1489

Elastic Legendre Coefficients

114-F1-289



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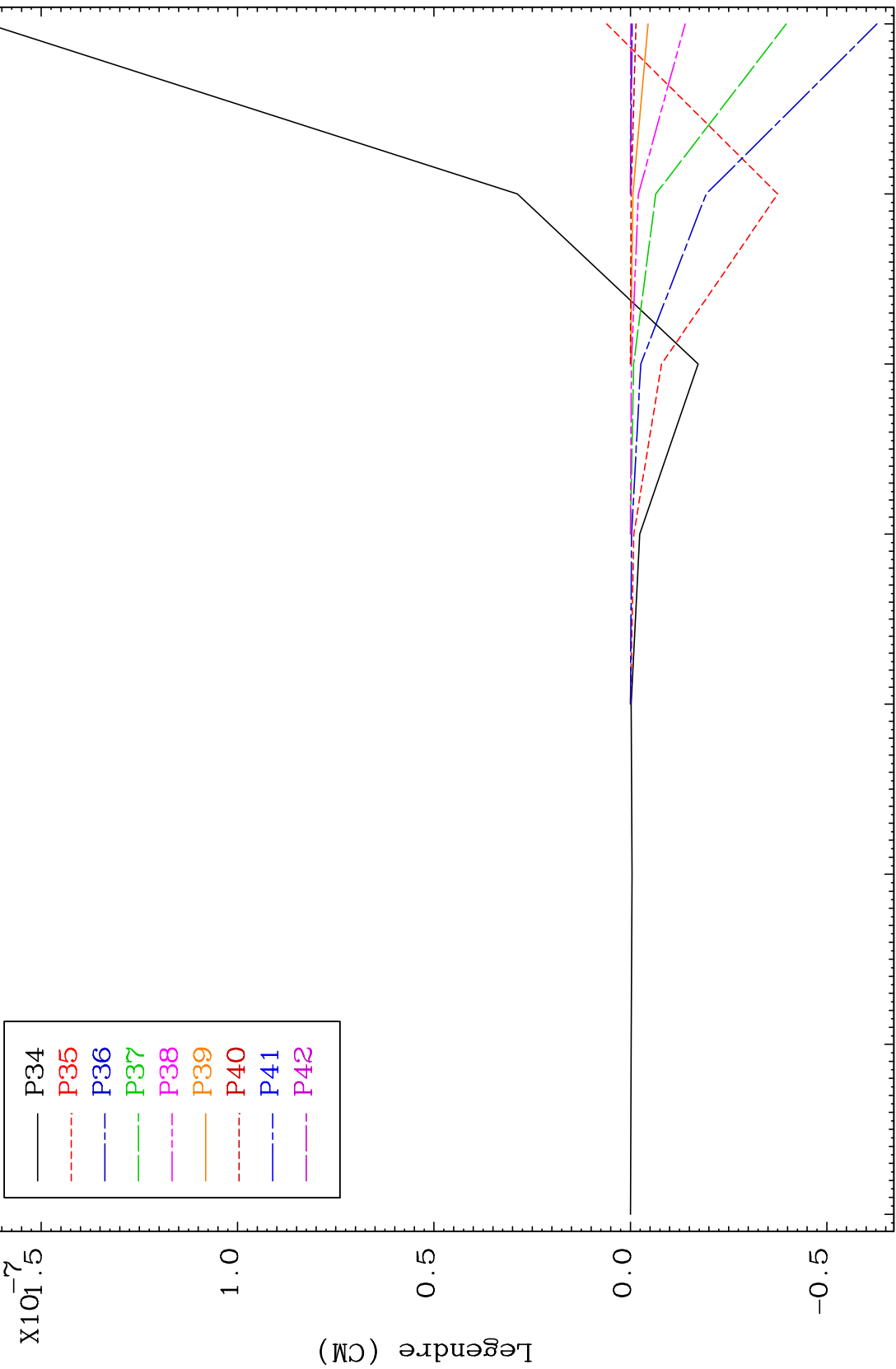
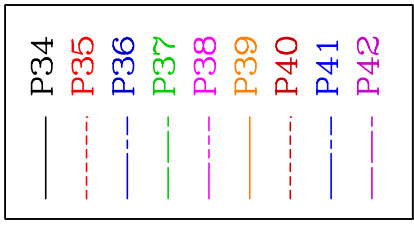
Incident Energy (MeV)

114-F1-289

MAT 1489

Elastic Legendre Coefficients

114-F1-289

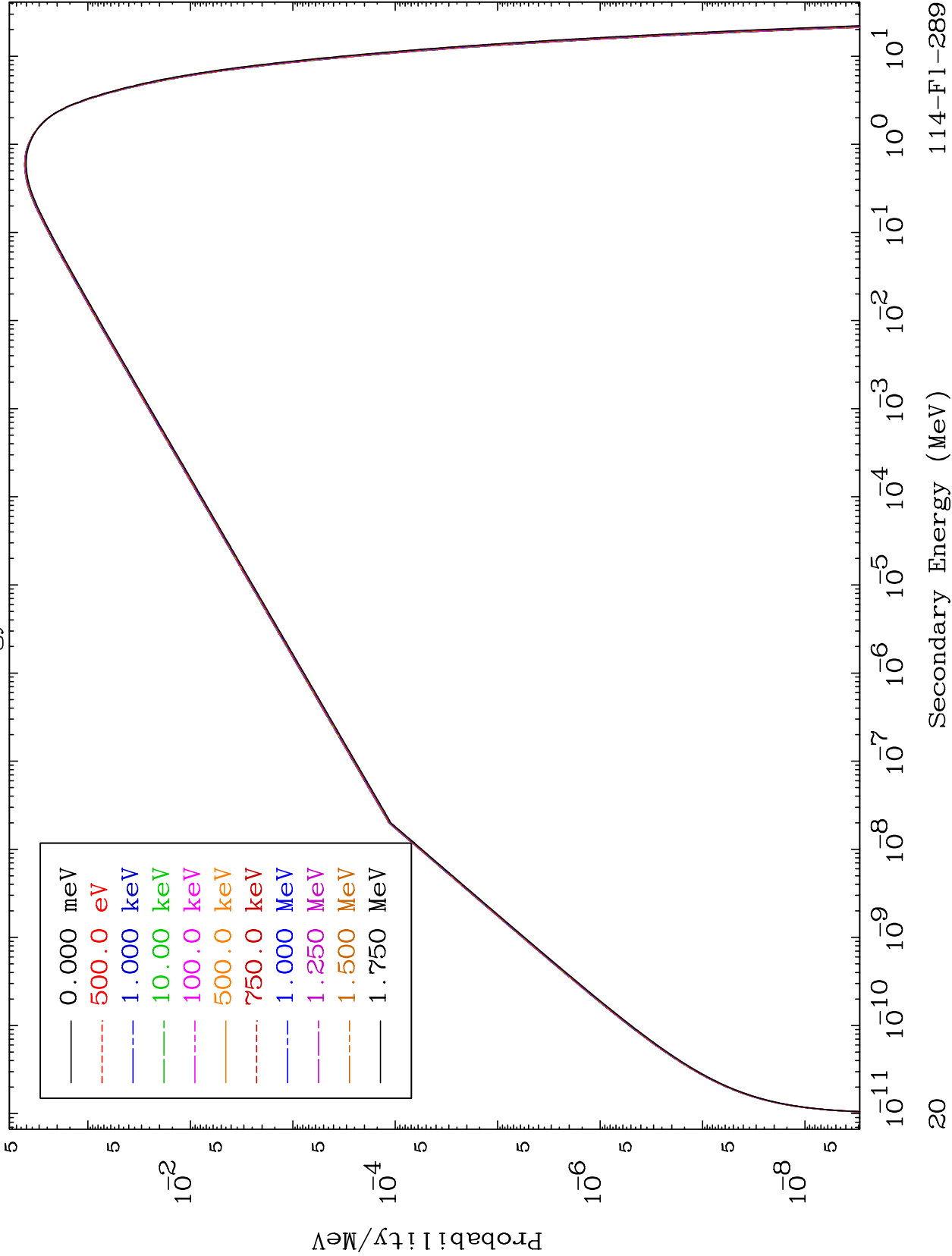


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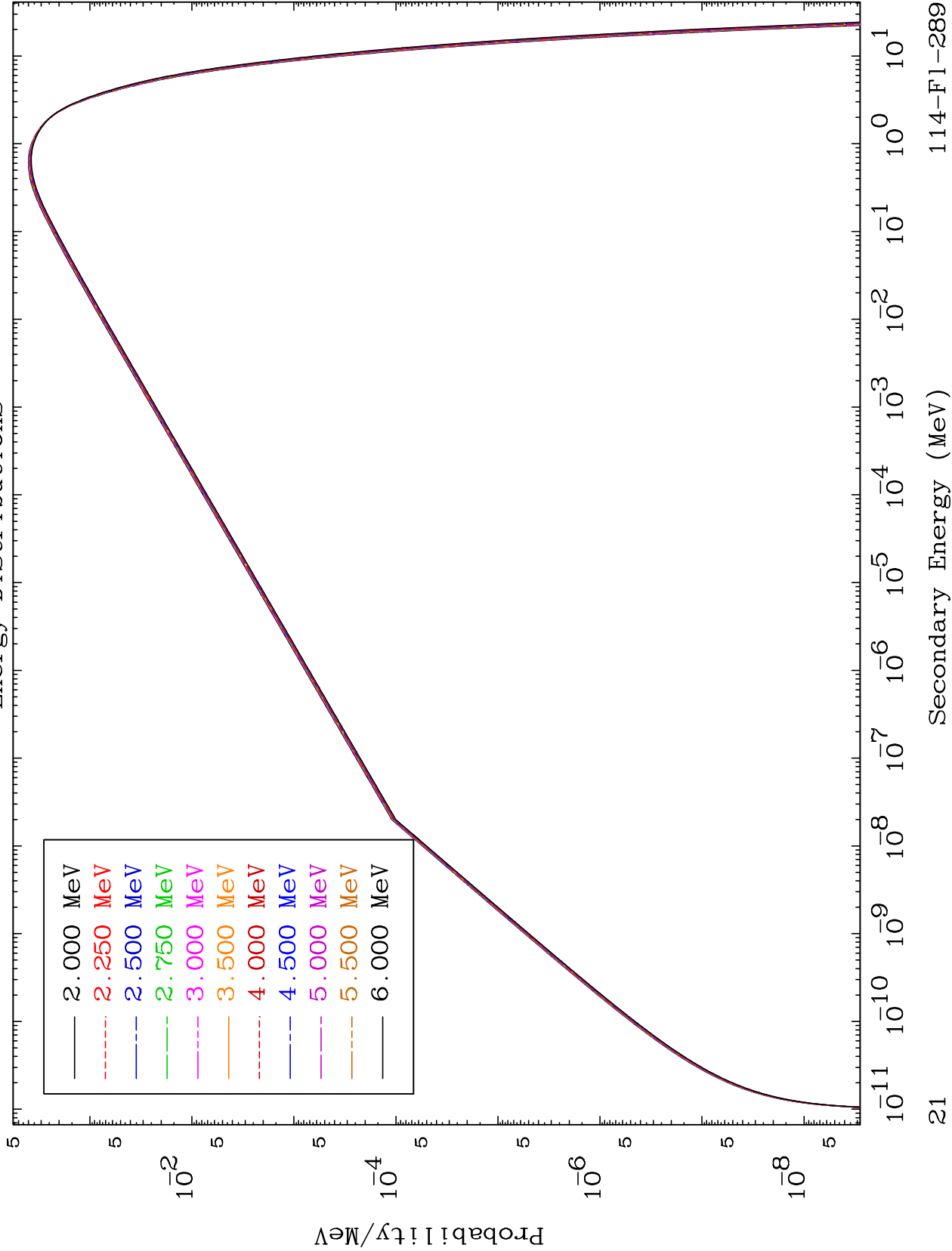
Incident Energy (MeV)

114-F1-289

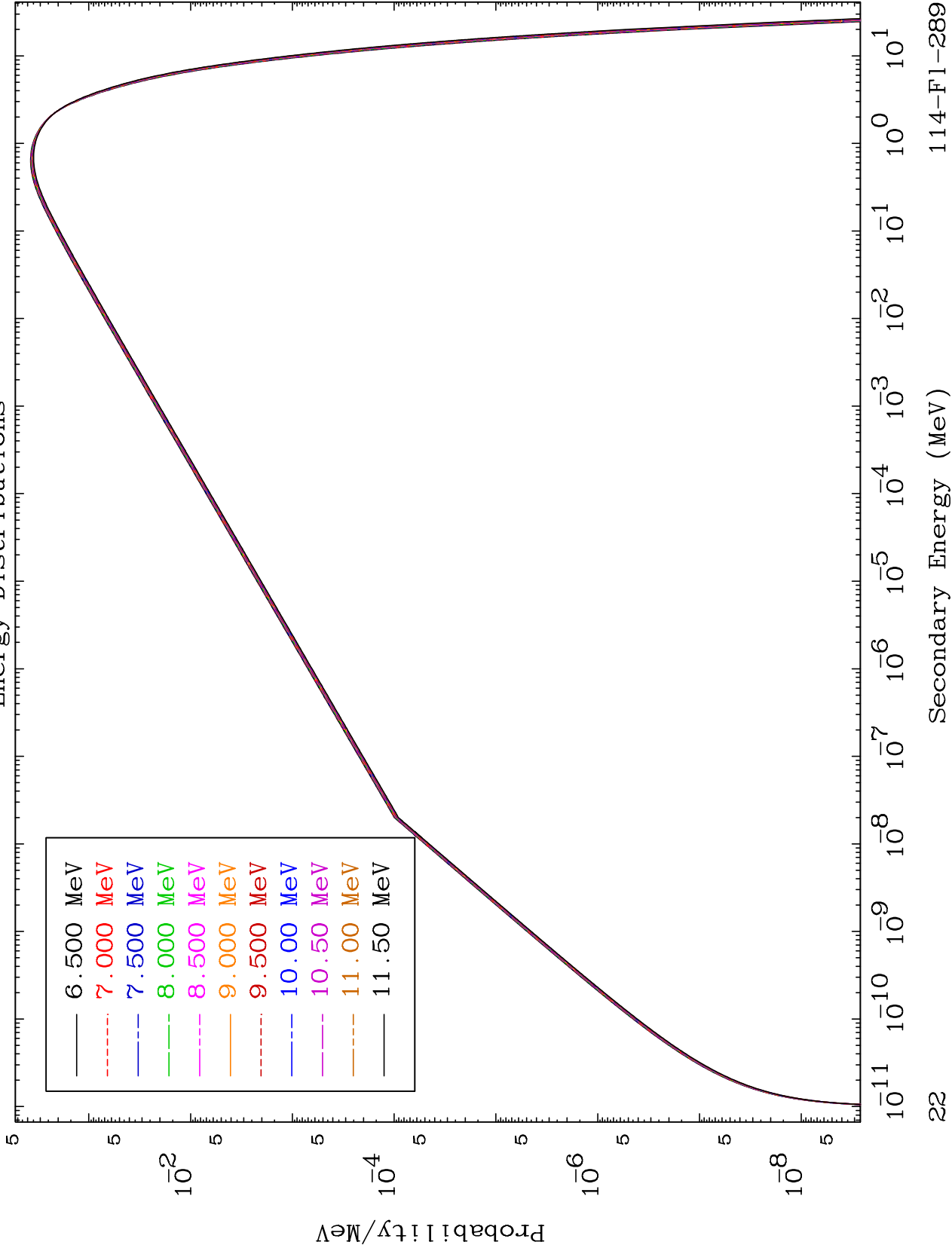
Fission Energy Distributions

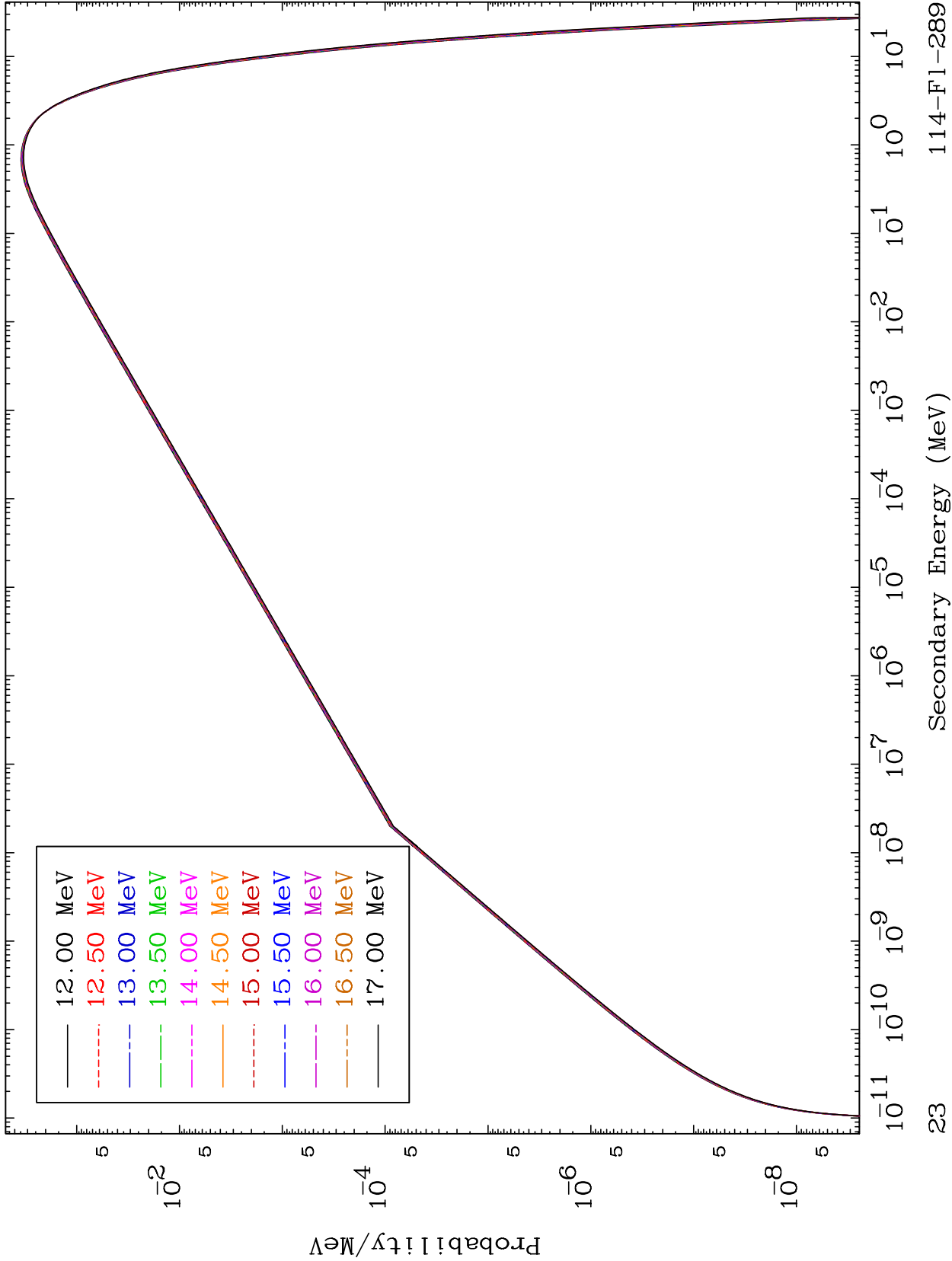


Fission
Energy Distributions



Fission Energy Distributions





Fission Energy Distributions

