

Program EVALPLOT
(Version 2021-1)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

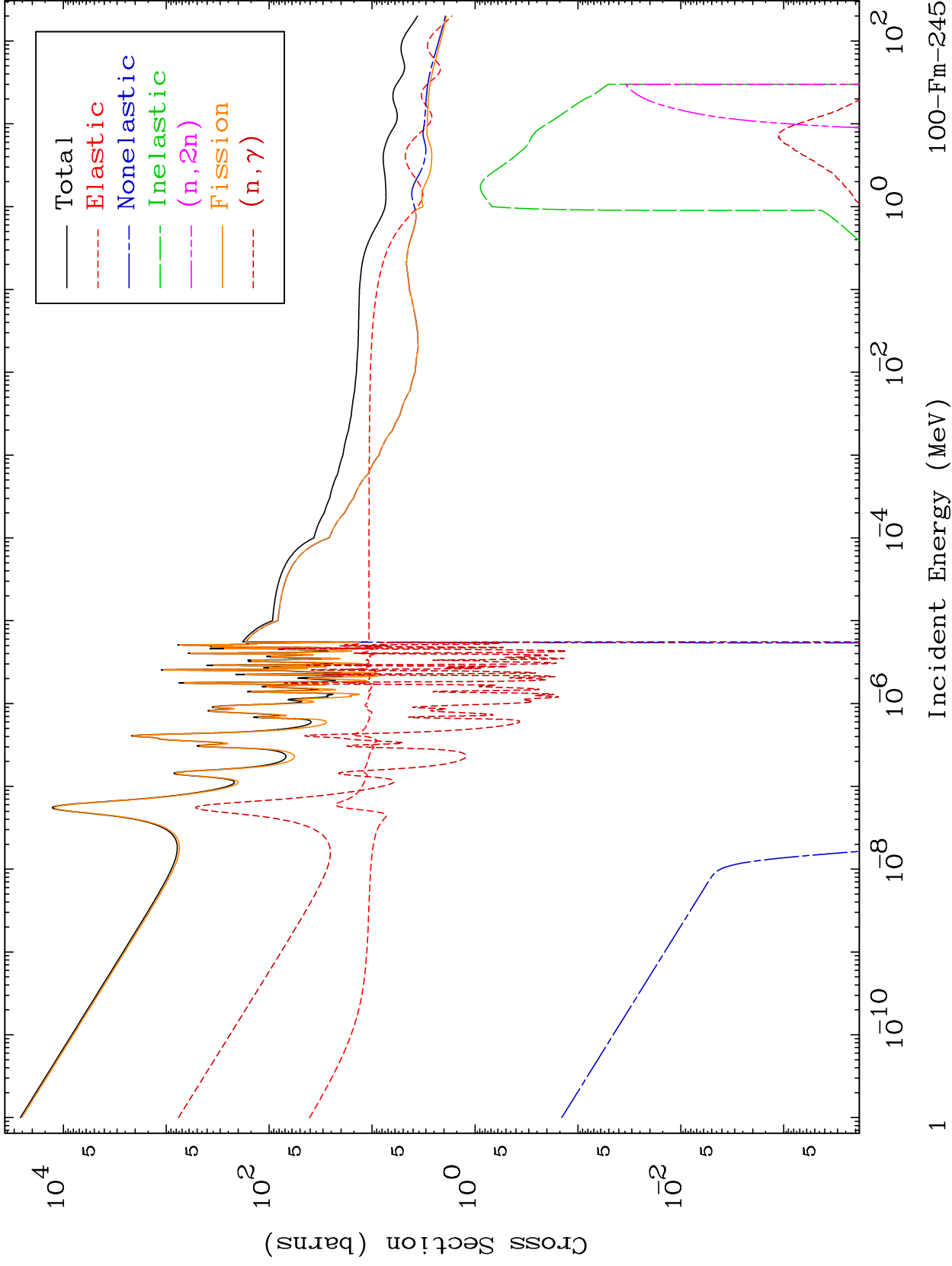
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Press Mouse Button to Start

MAT 9926

Neutron Major
293 Kelvin Cross Sections

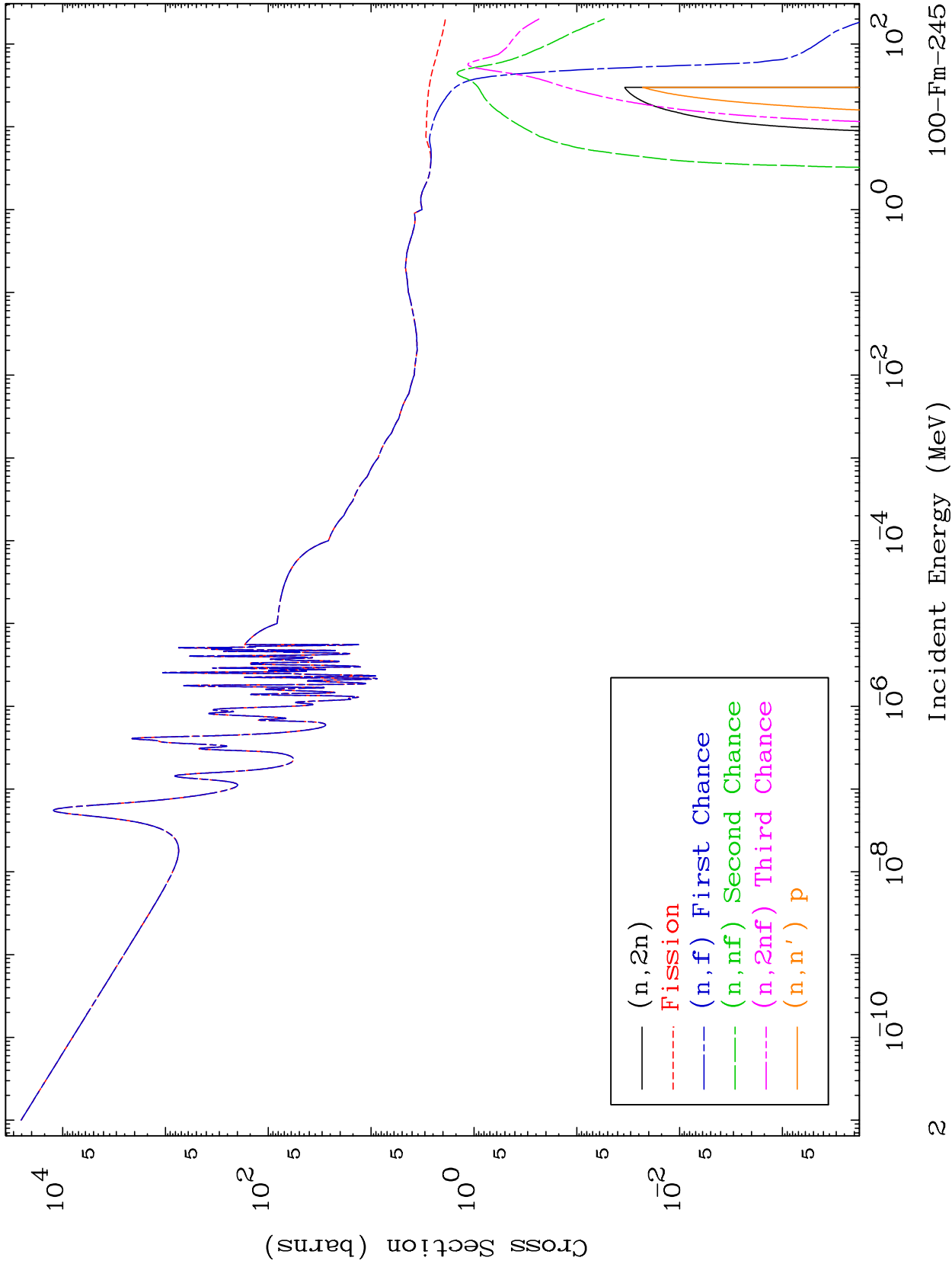
100-Fm-245

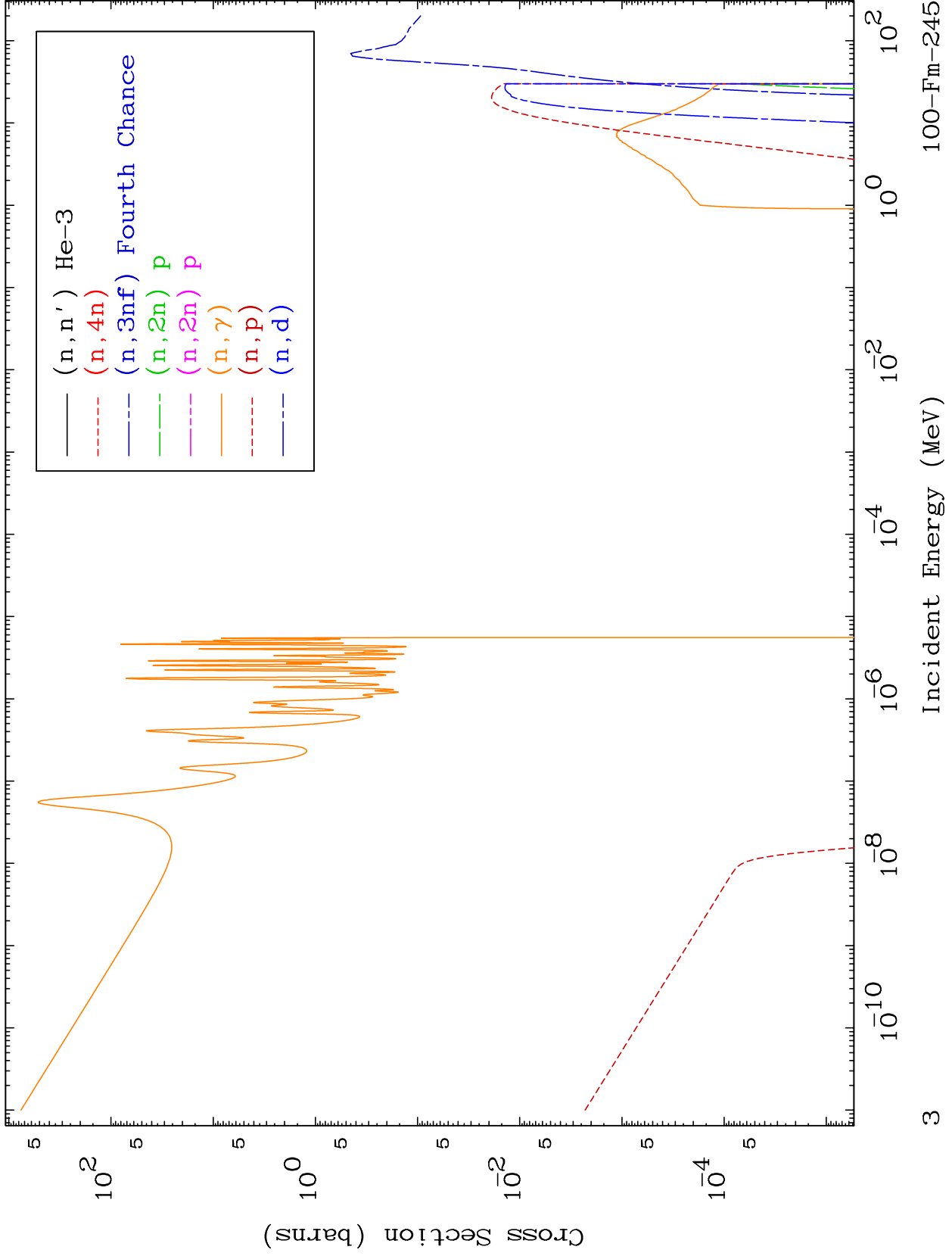


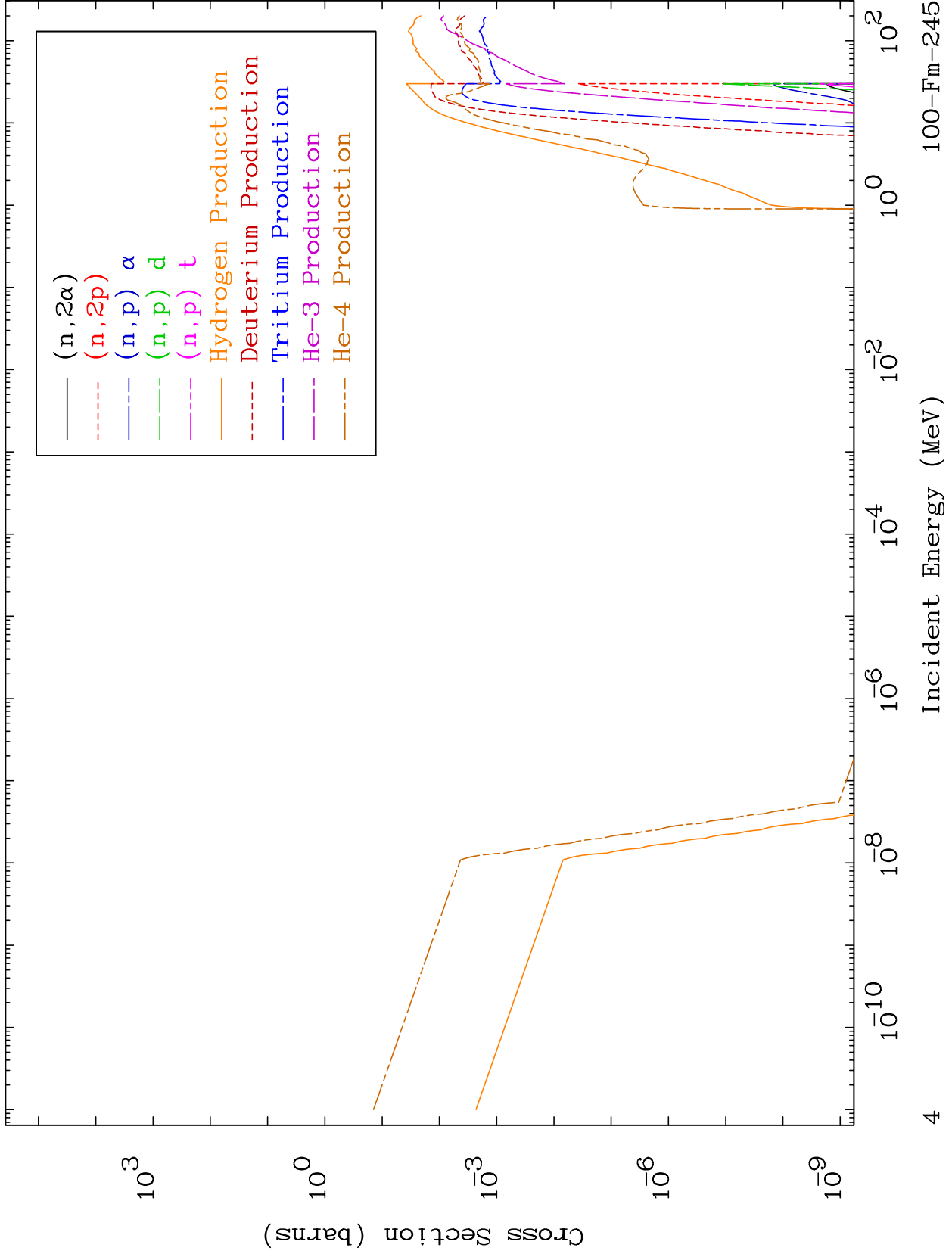
MAT 9926

Neutron Absorption
293 Kelvin Cross Sections

100-Fm-245



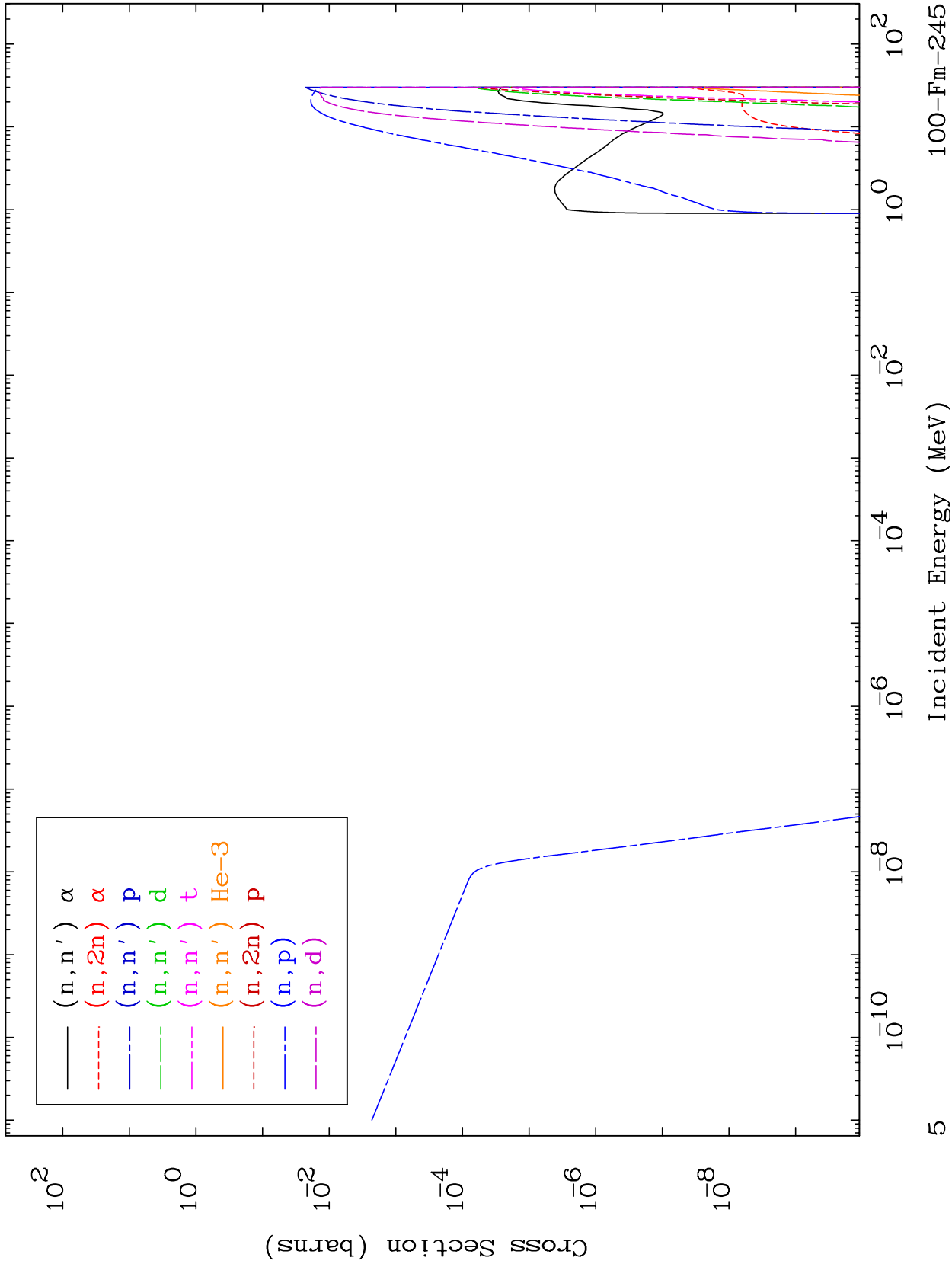




MAT 9926

Charged Particle
293 Kelvin Cross Sections

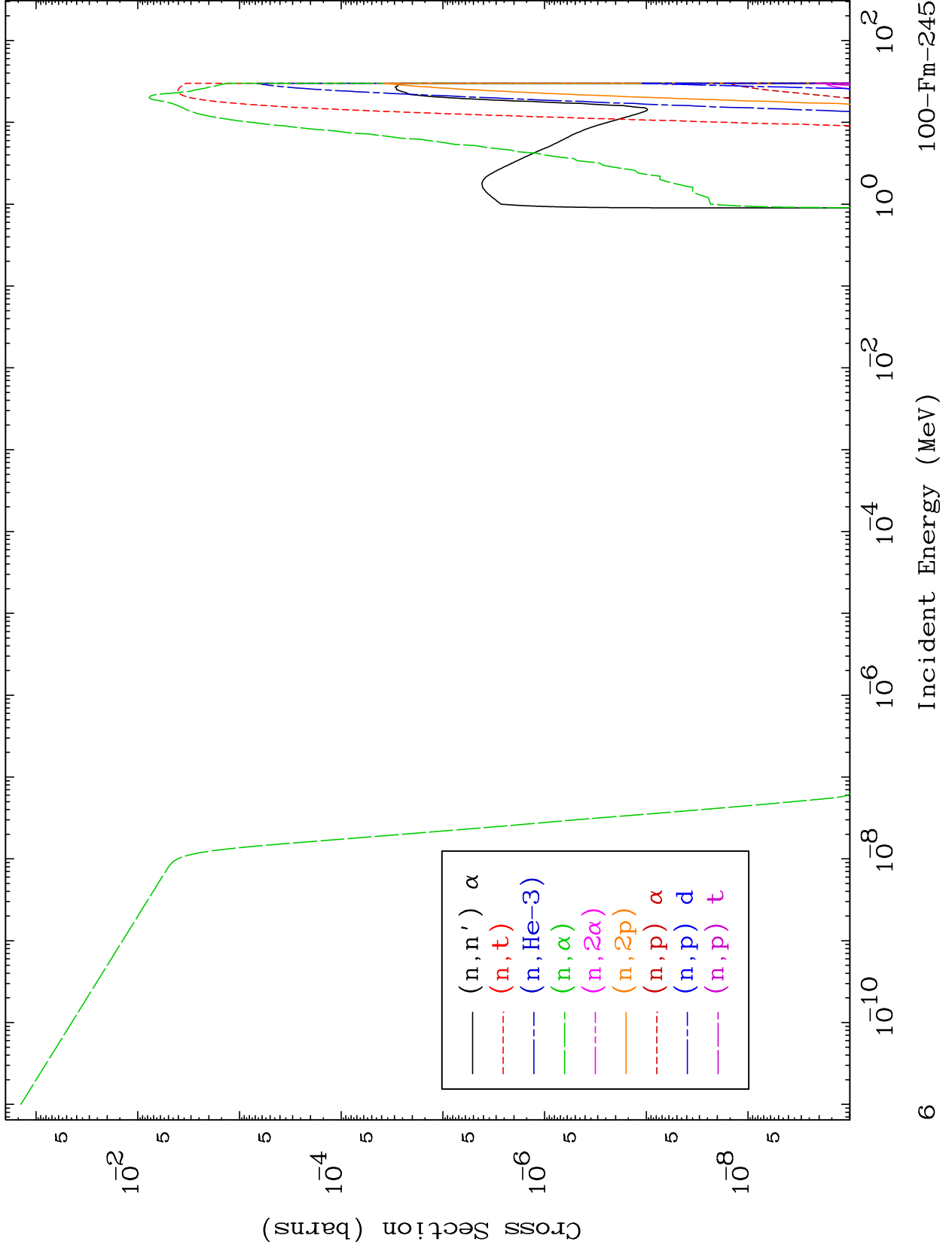
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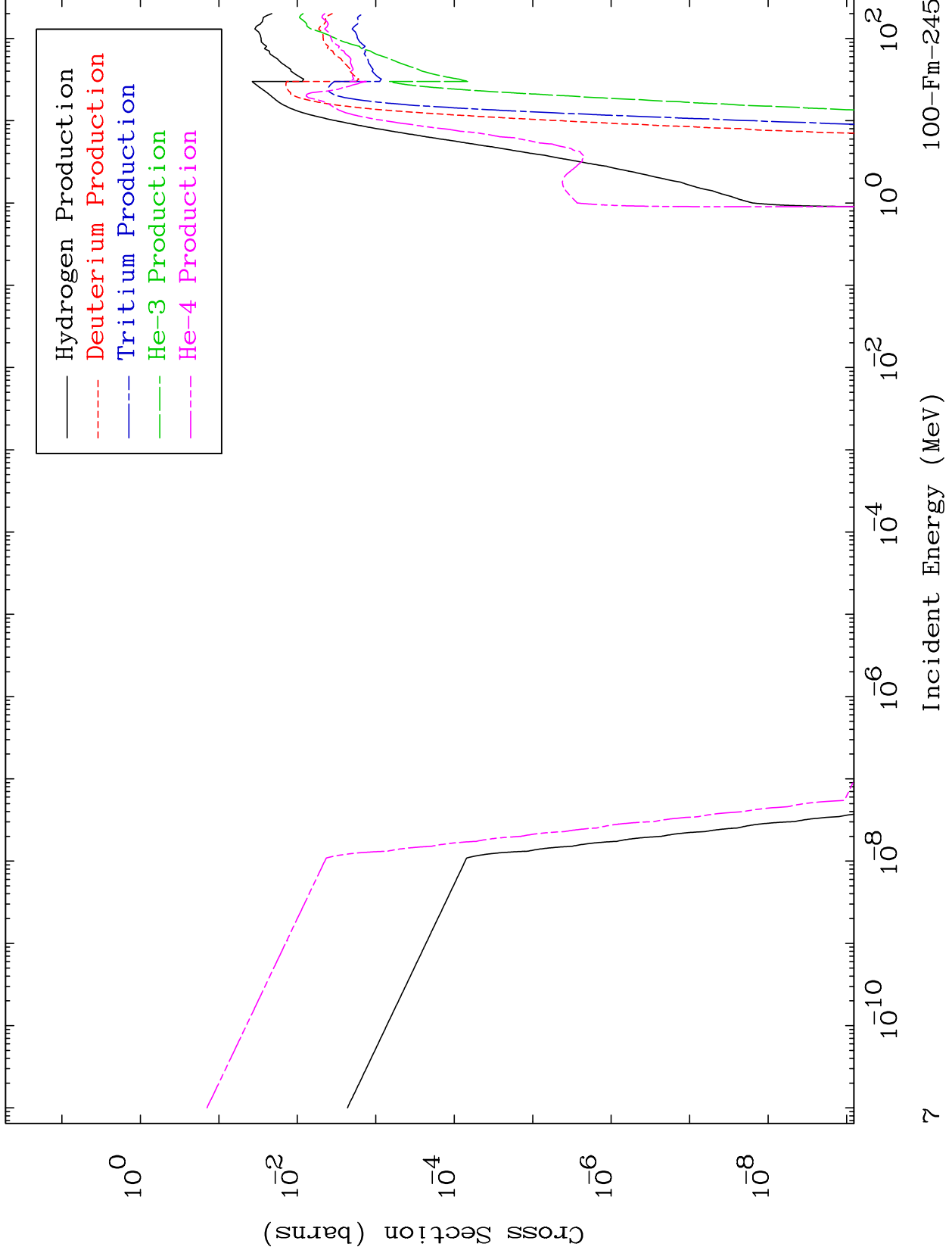


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Charged Particle
293 Kelvin Cross Sections

100-Fm-245

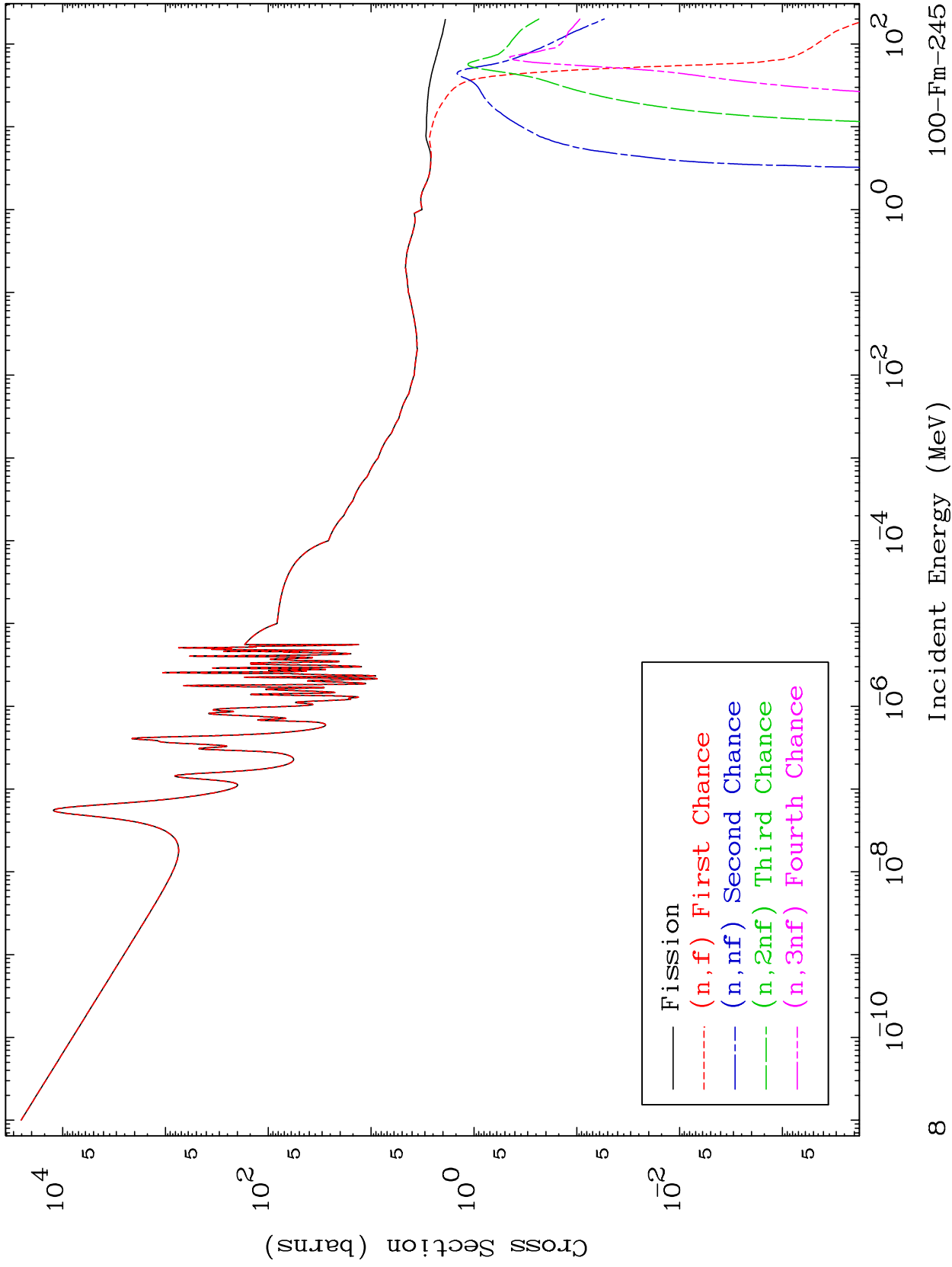




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Fission
293 Kelvin Cross Sections

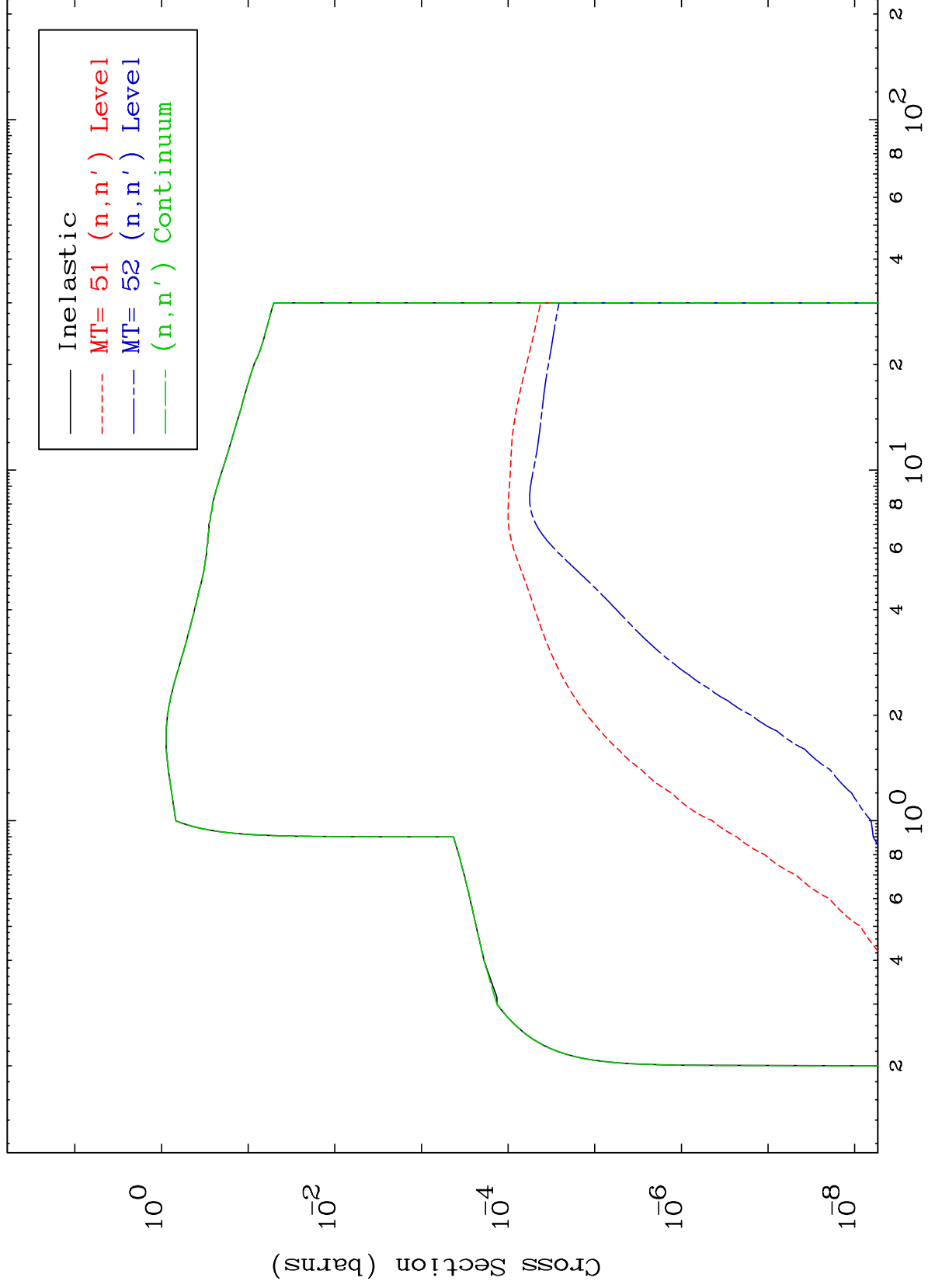
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(n,n') Levels
293 Kelvin Cross Sections

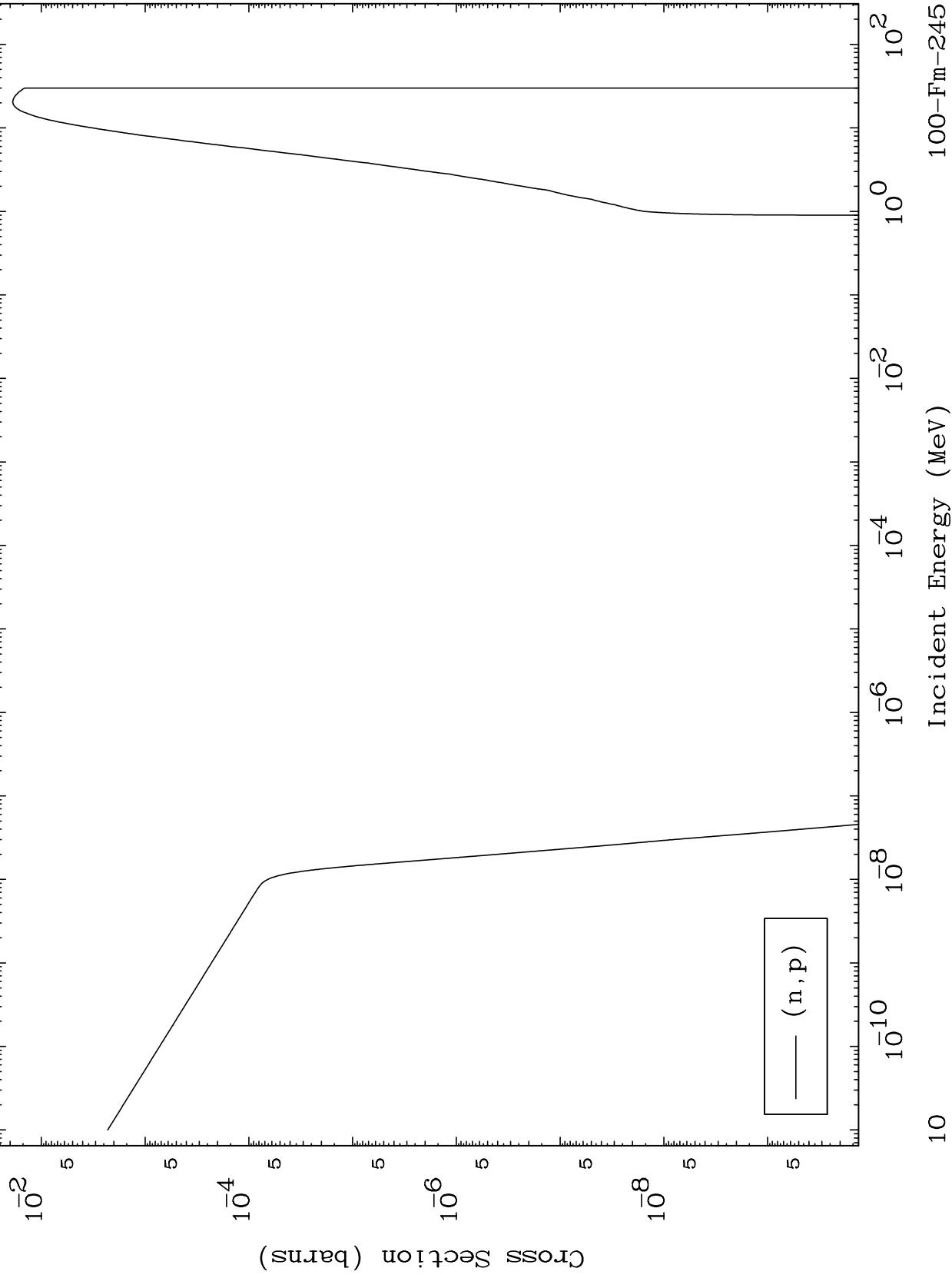
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(n,p) Levels
293 Kelvin Cross Sections

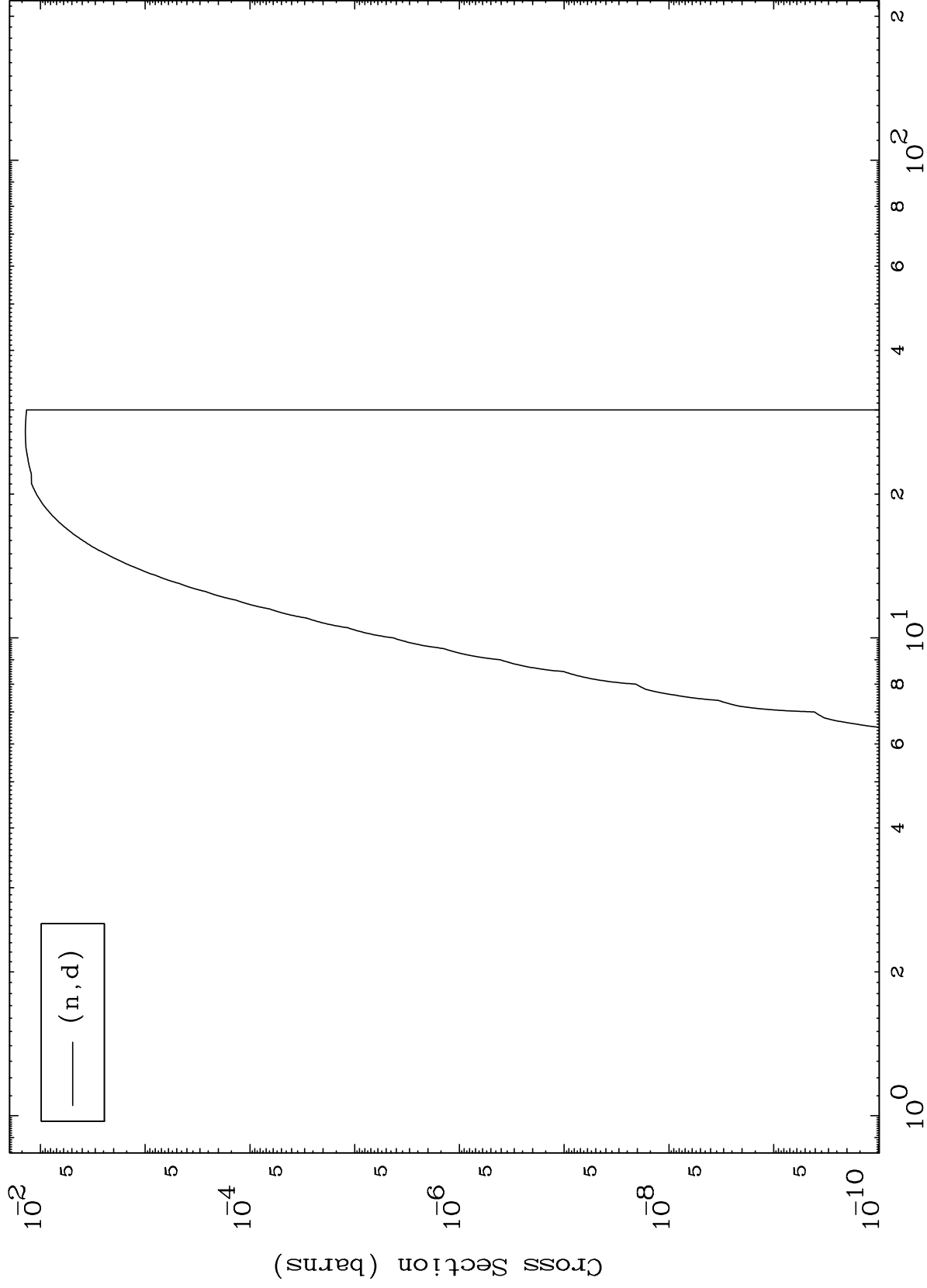
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(n,d) Levels
293 Kelvin Cross Sections

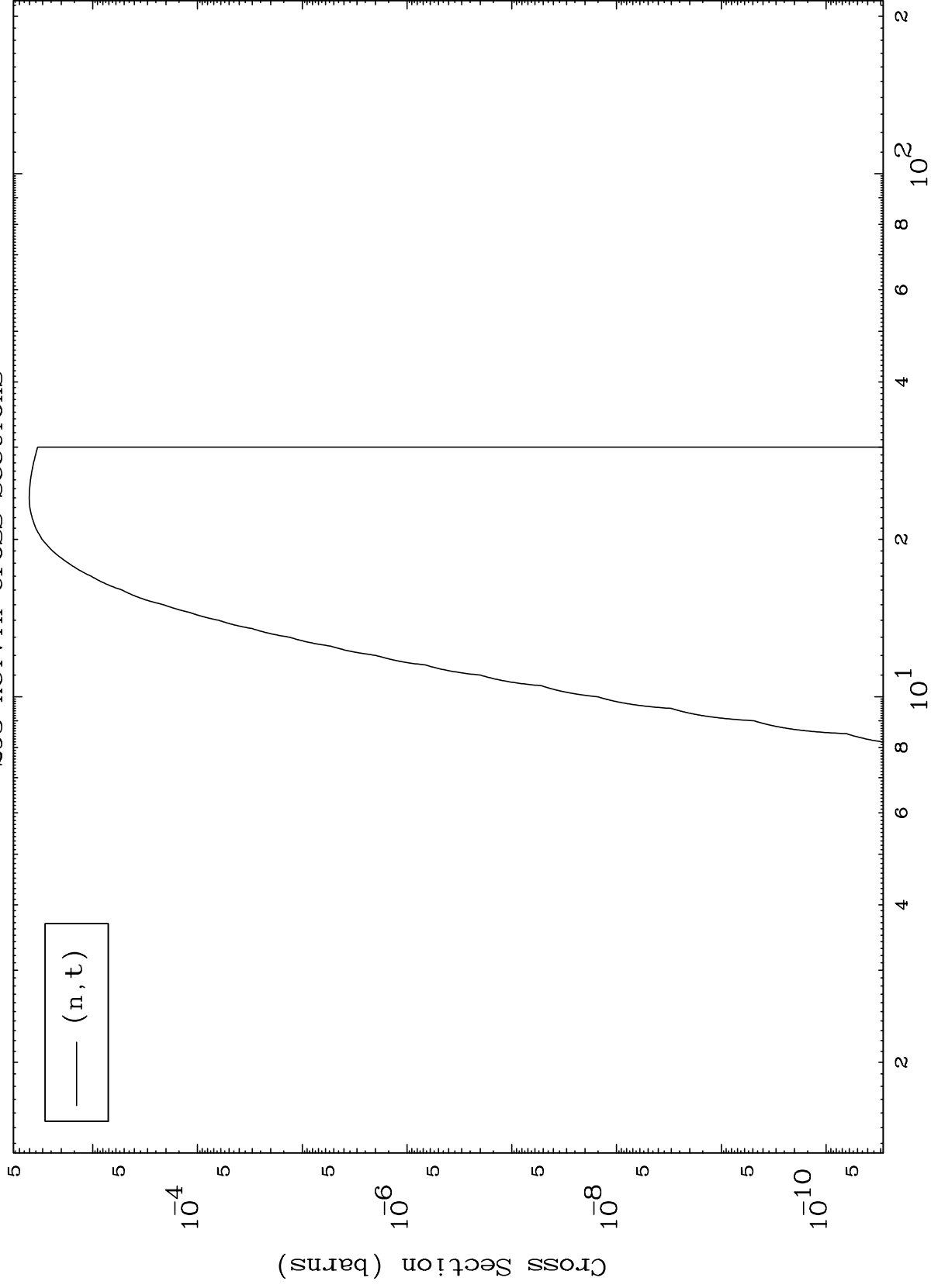
100-Fm-245



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(n,t) Levels
293 Kelvin Cross Sections

100-Fm-245



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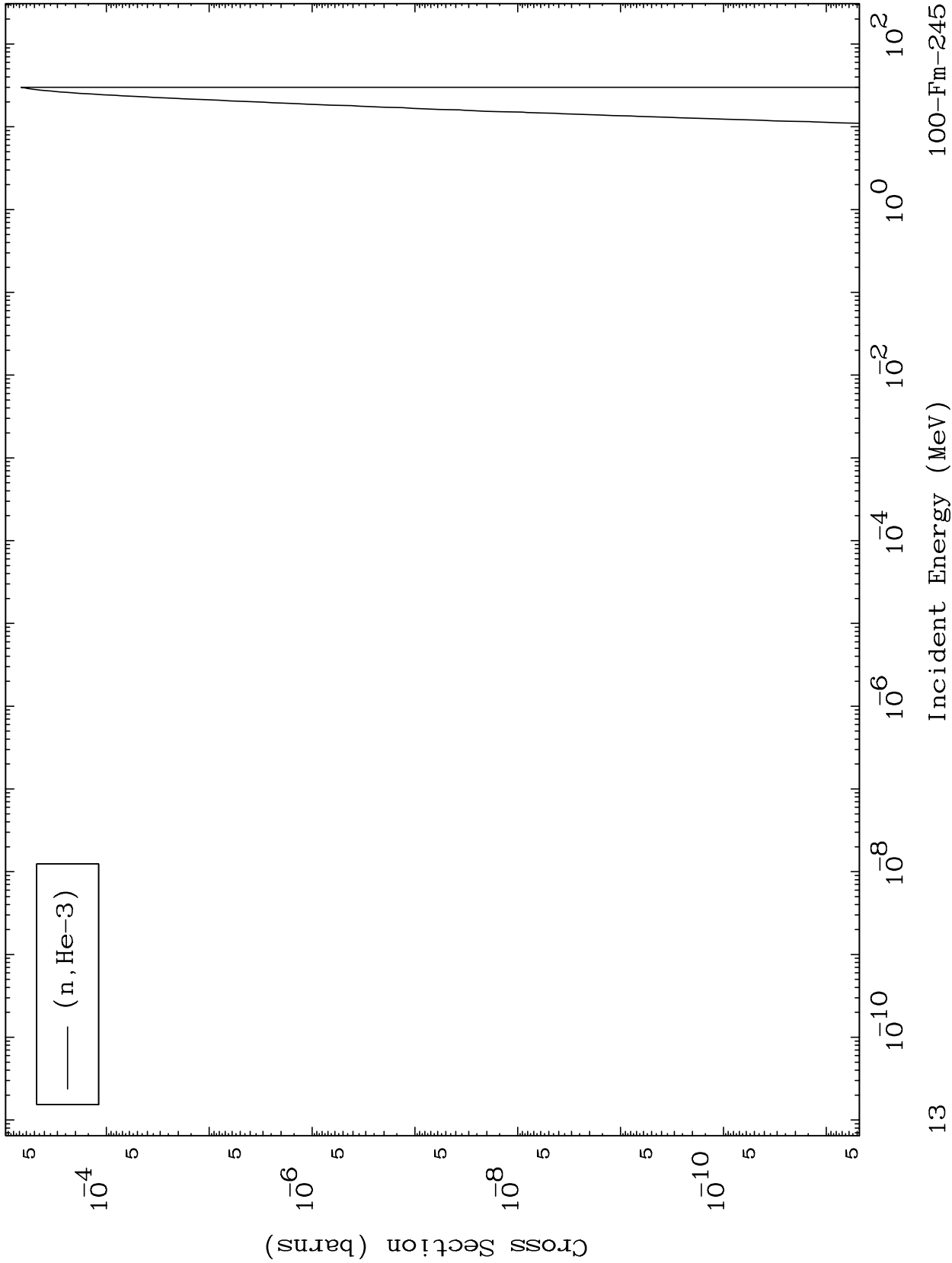
Incident Energy (MeV)

100-Fm-245

MAT 9926

(n,He3) Levels
293 Kelvin Cross Sections

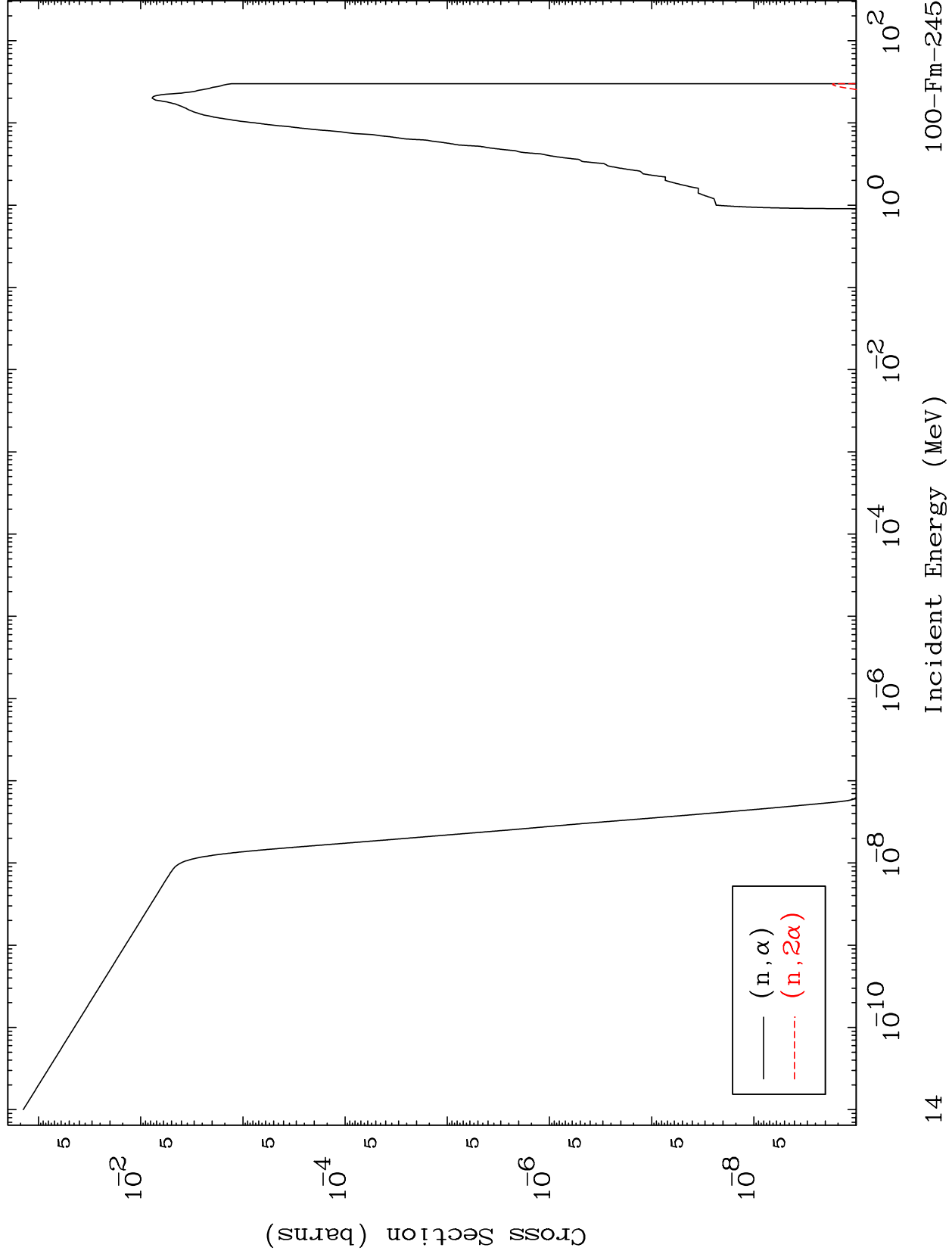
100-Fm-245



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(n, α) Levels
293 Kelvin Cross Sections

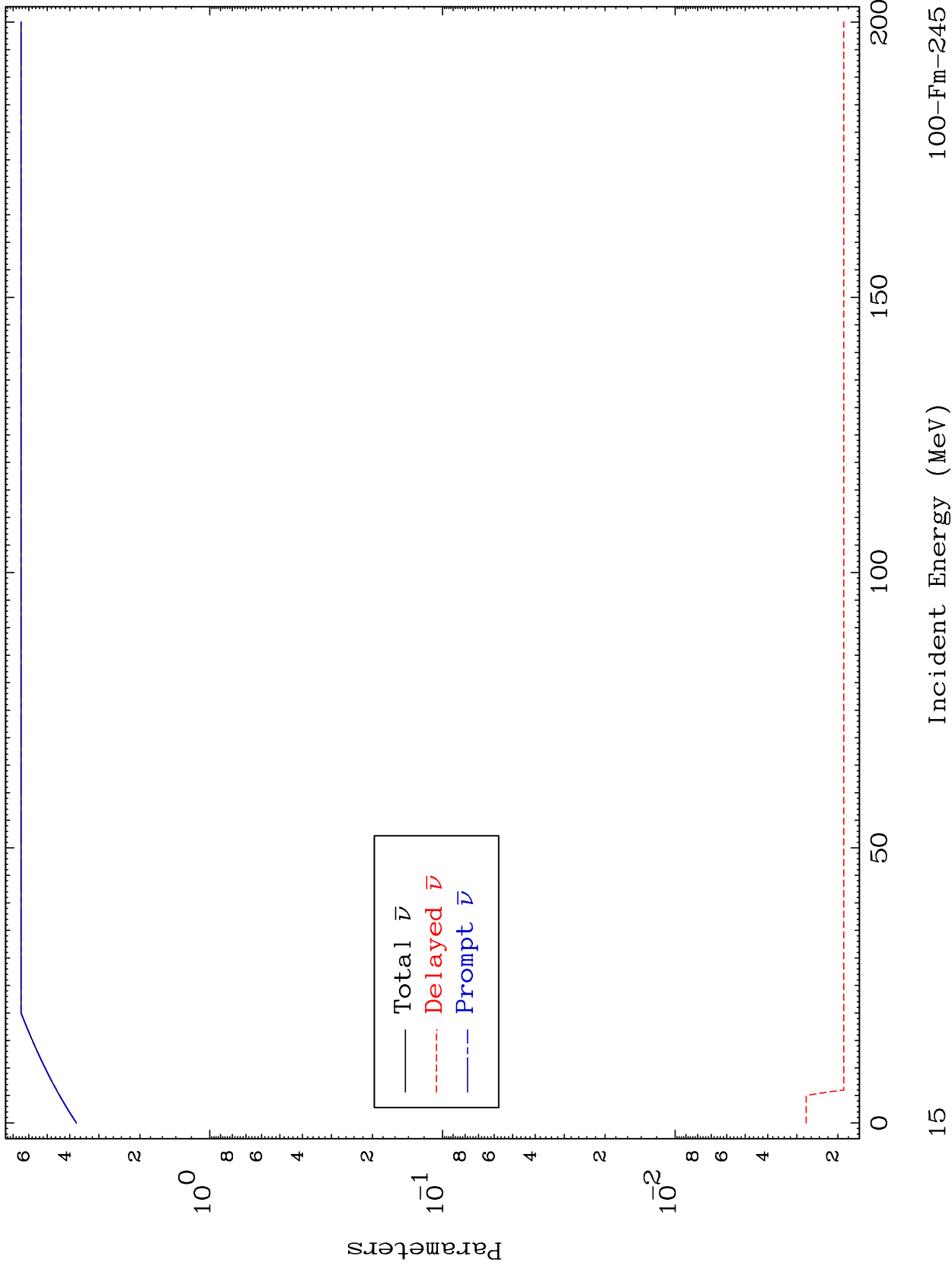
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Energy Release
Parameters

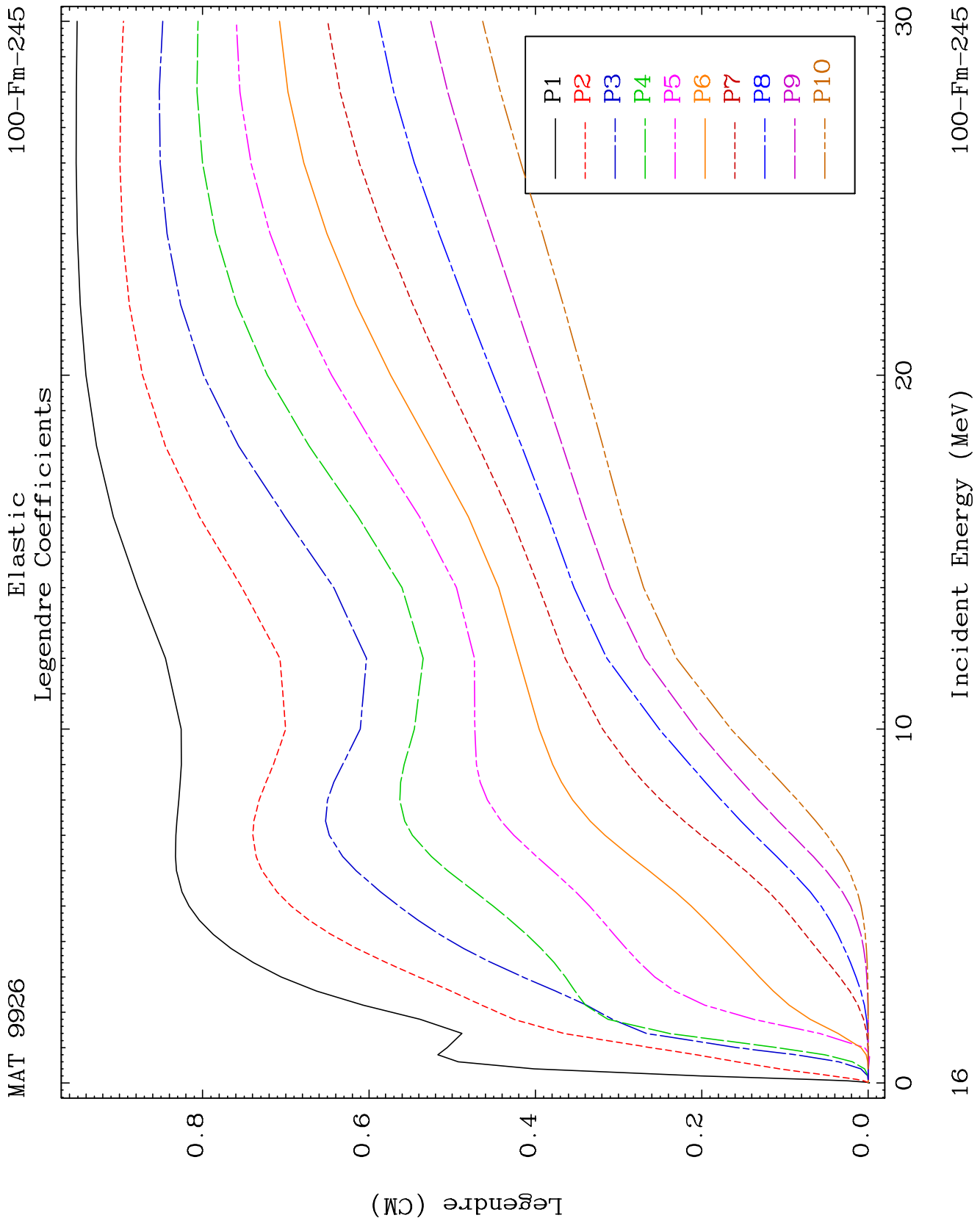
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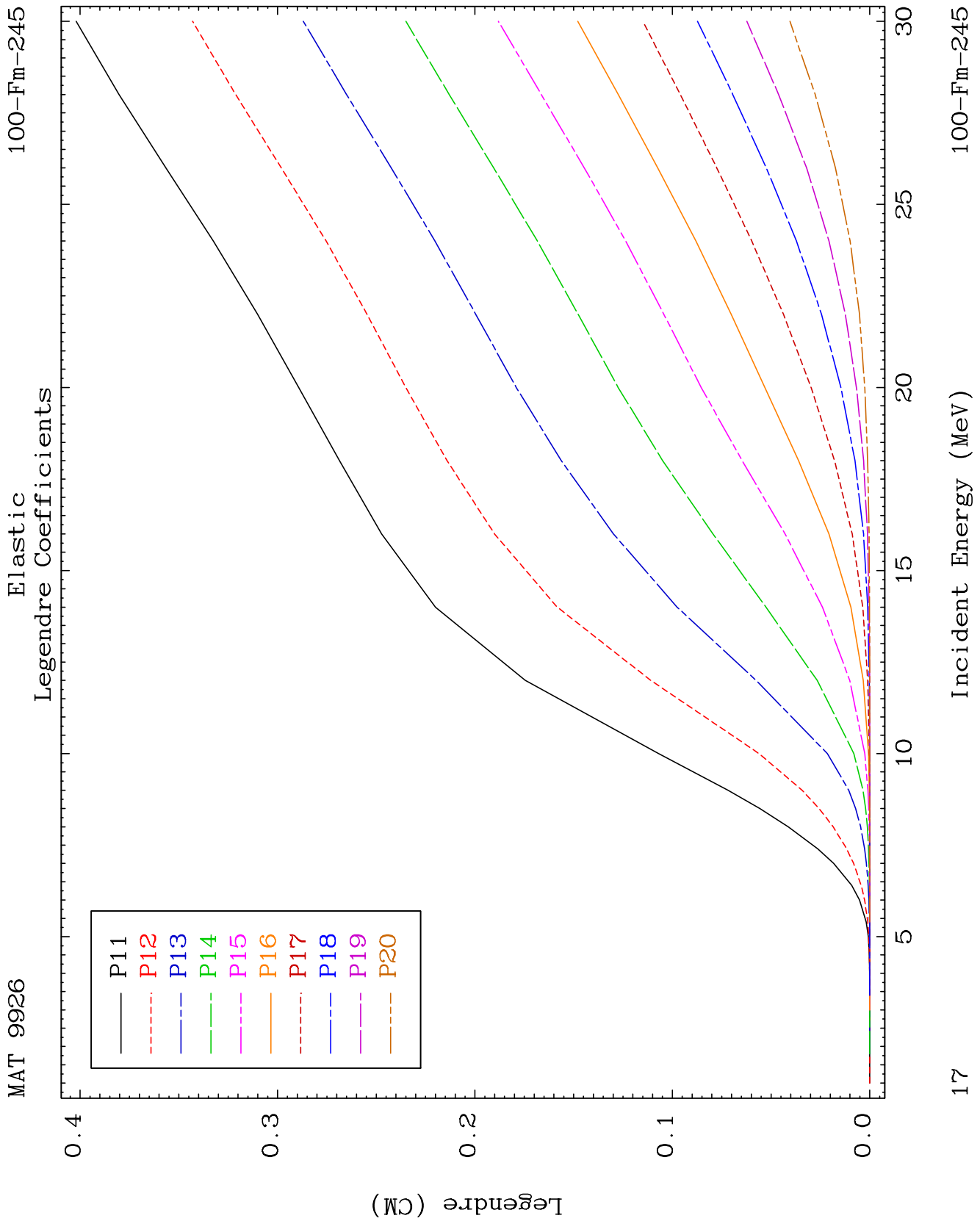


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Incident Energy (MeV)

100-Fm-245

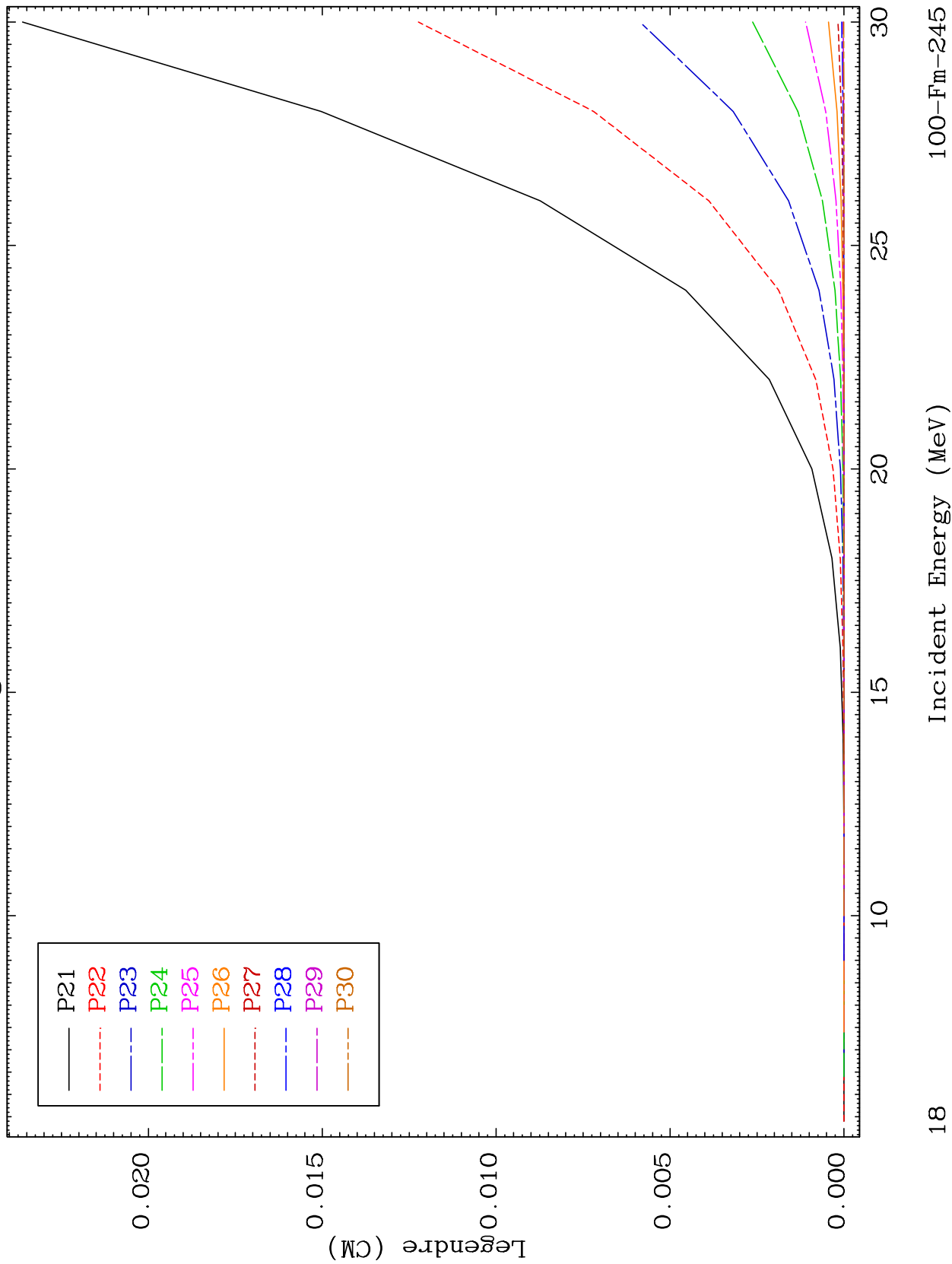




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Elastic Legendre Coefficients

100-Fm-245



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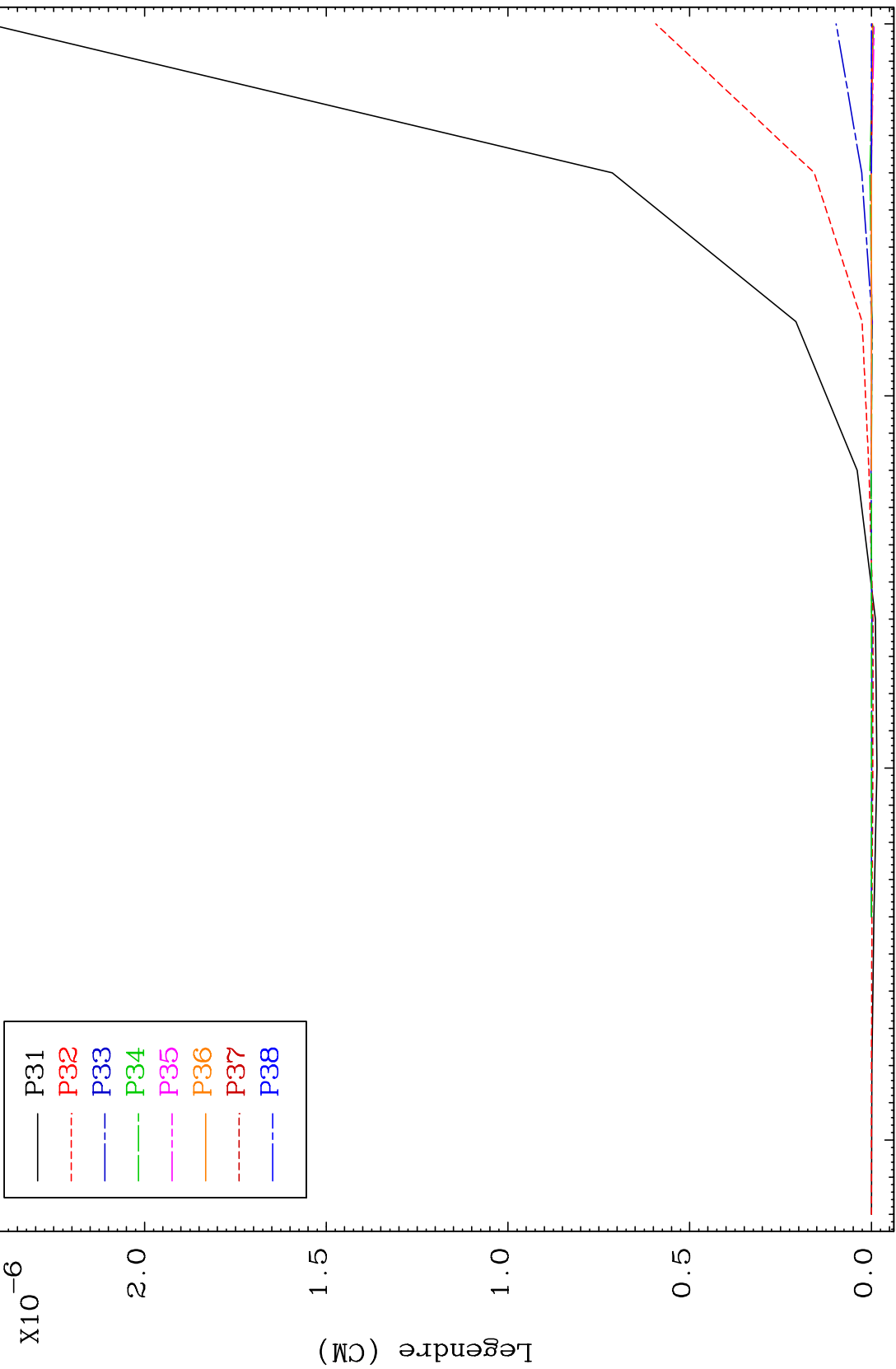
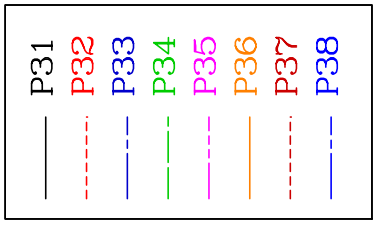
Incident Energy (MeV)

100-Fm-245

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Elastic Legendre Coefficients

100-Fm-245



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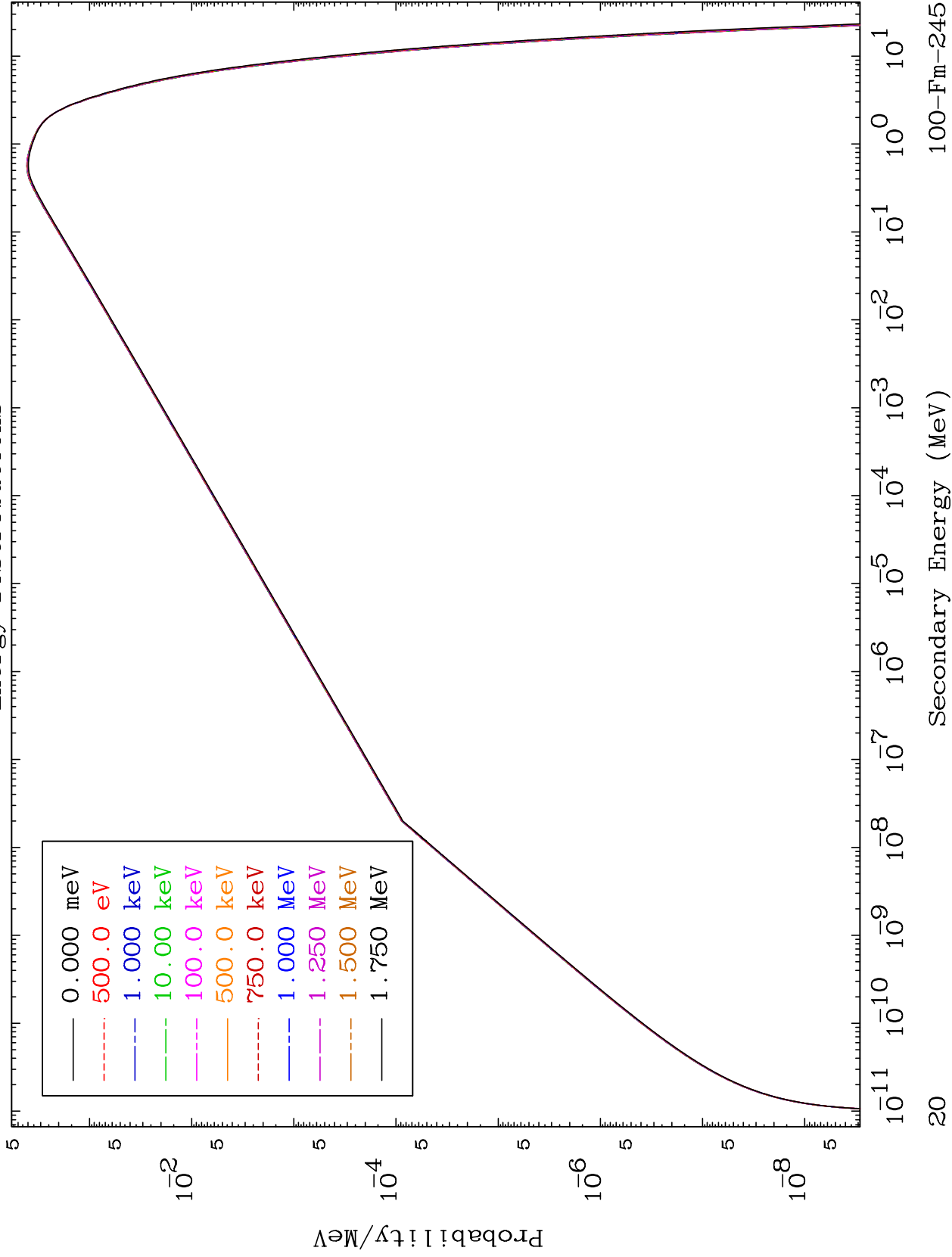
Incident Energy (MeV)

100-Fm-245

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Fission Energy Distributions

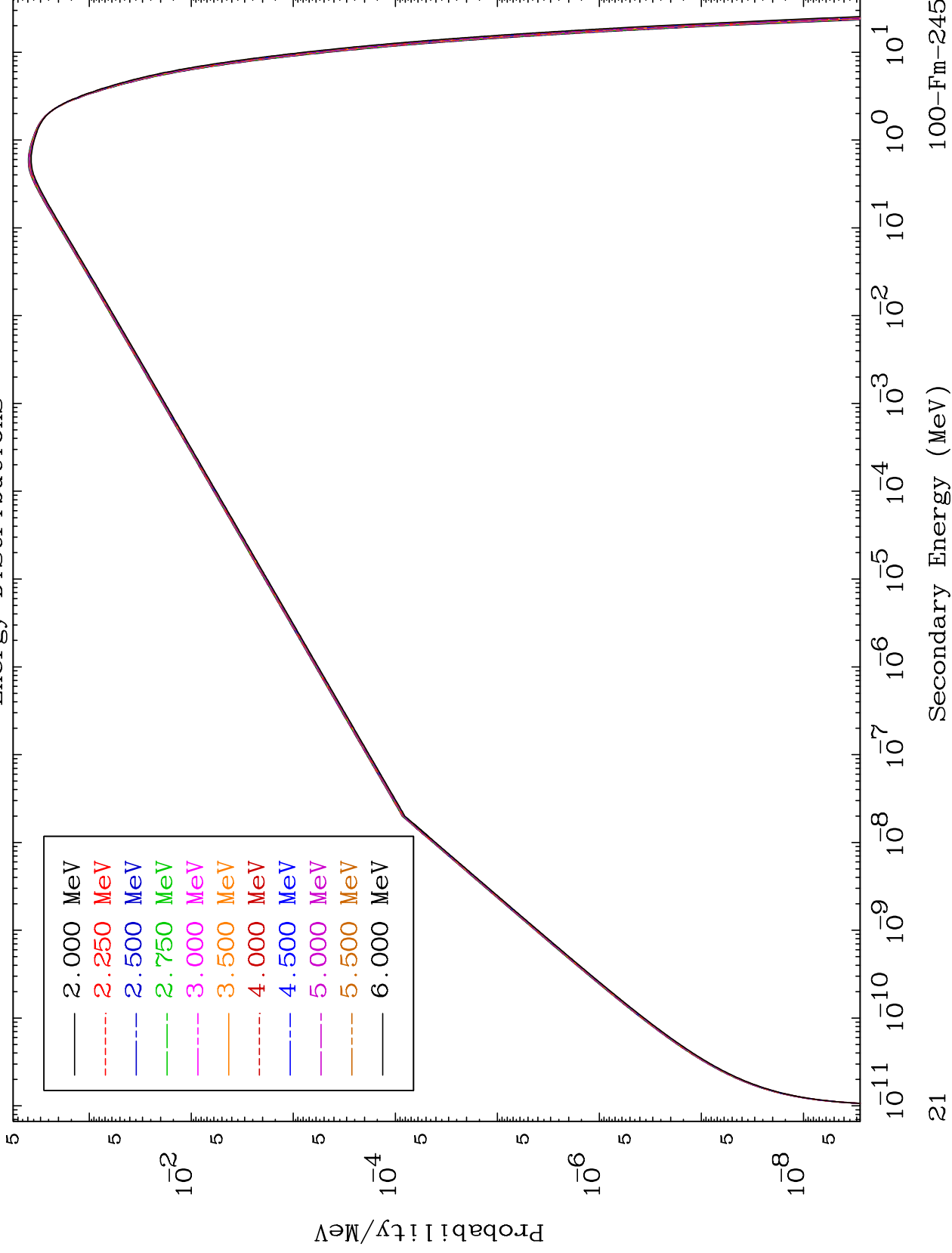
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Fission Energy Distributions

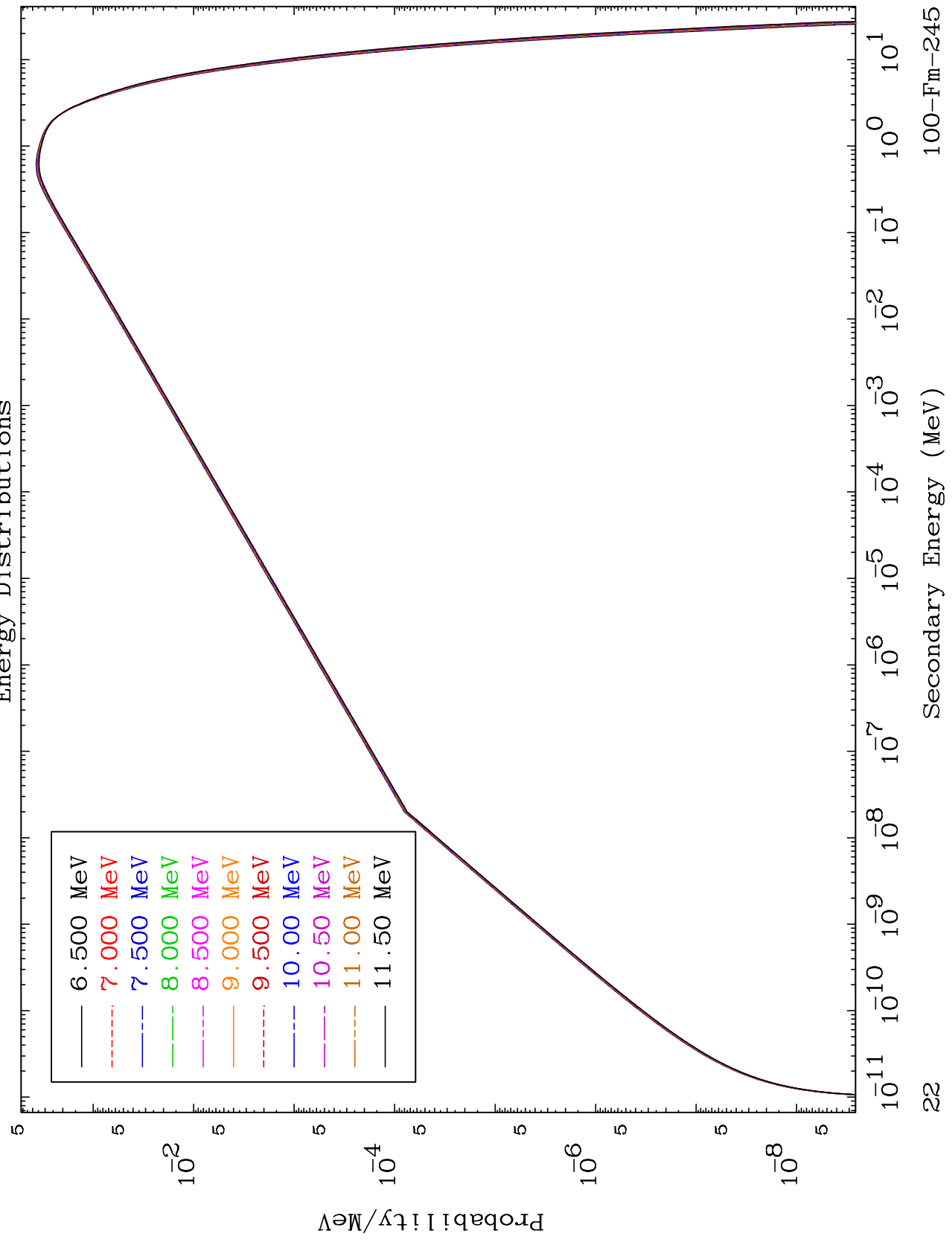
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Fission Energy Distributions

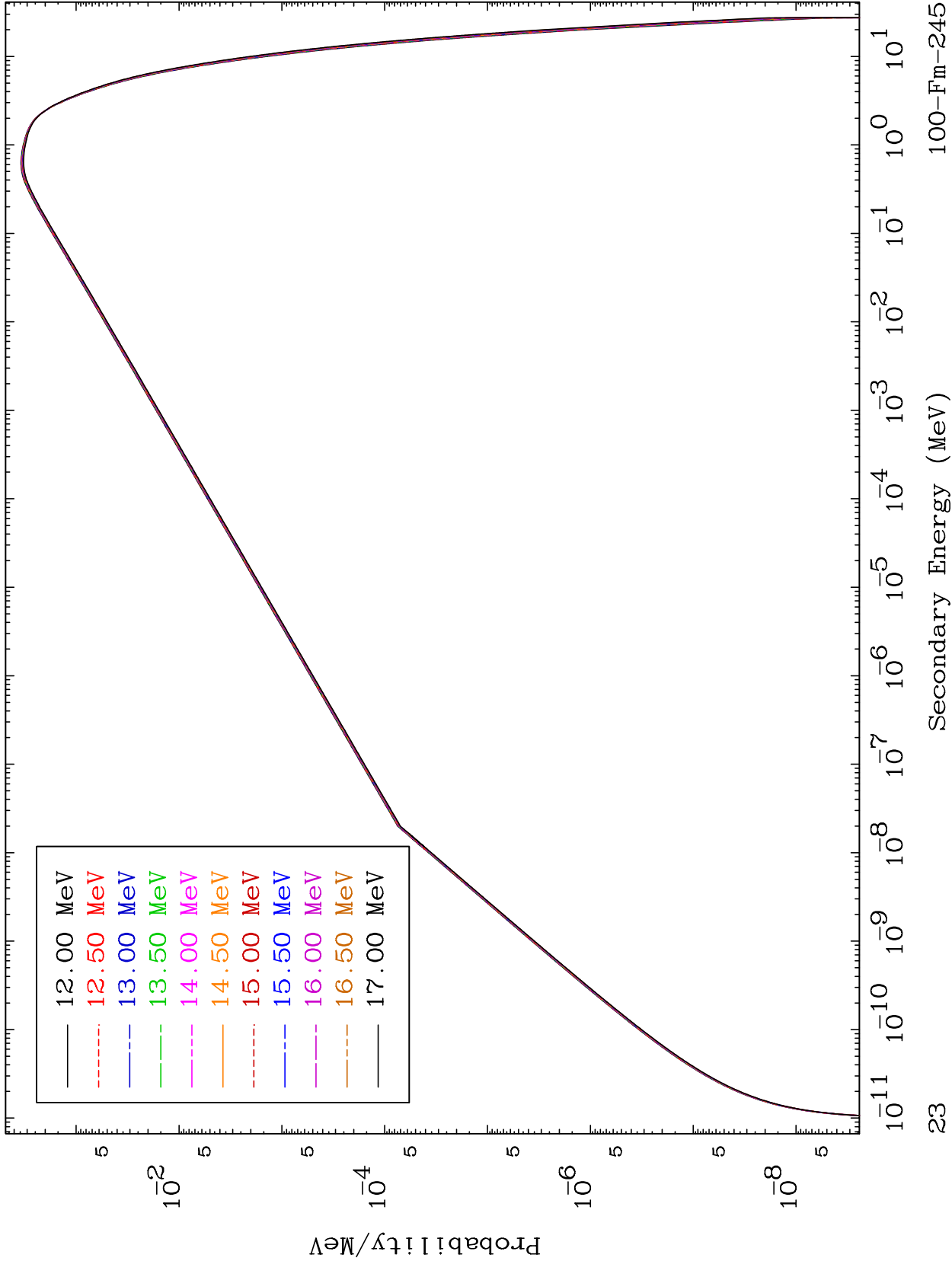
100-Fm-245



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Fission Energy Distributions

100-Fm-245



Fission Energy Distributions

100-Fm-245

