

Program EVALPLOT
(Version 2021-1)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

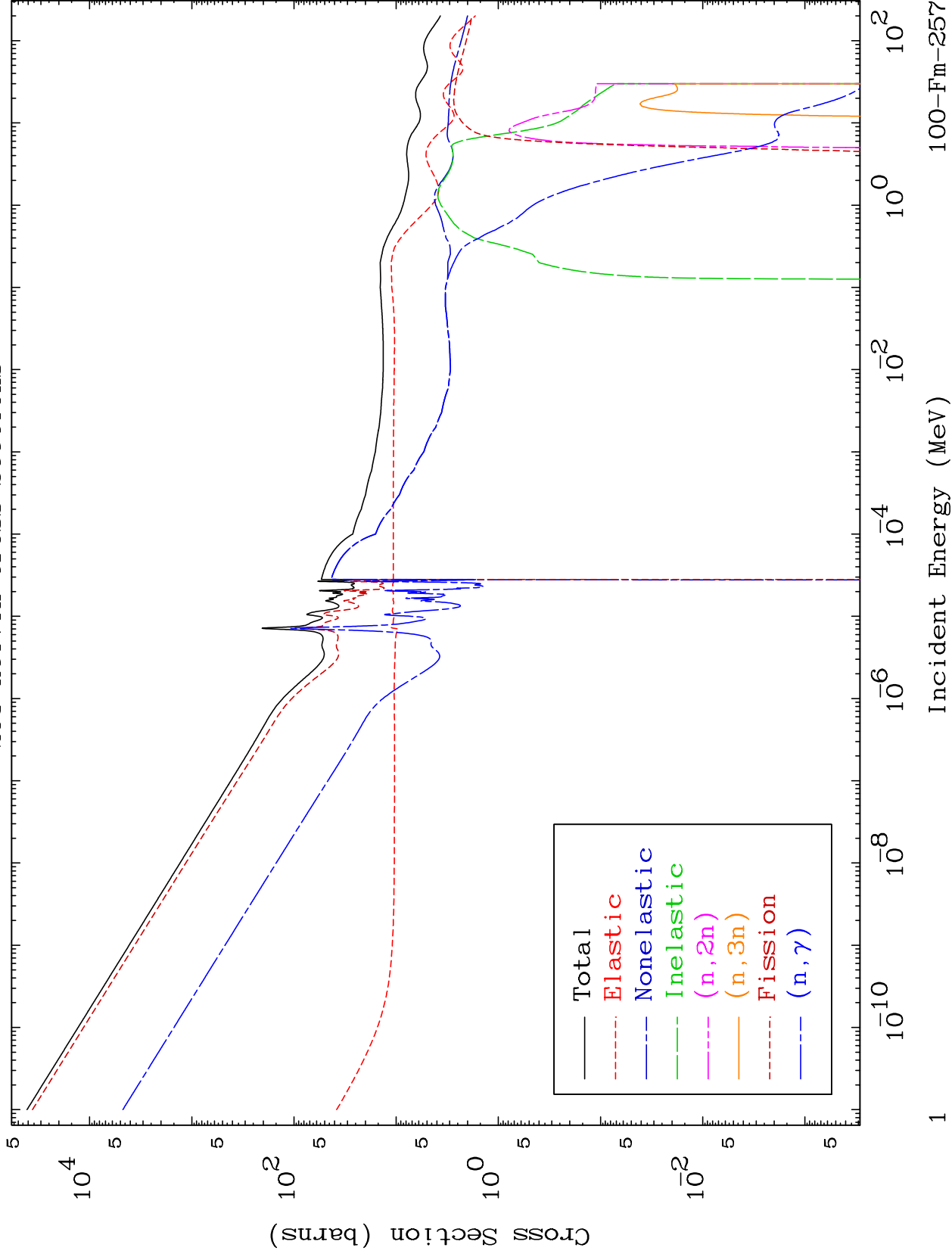
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9938

Neutron Major
293 Kelvin Cross Sections

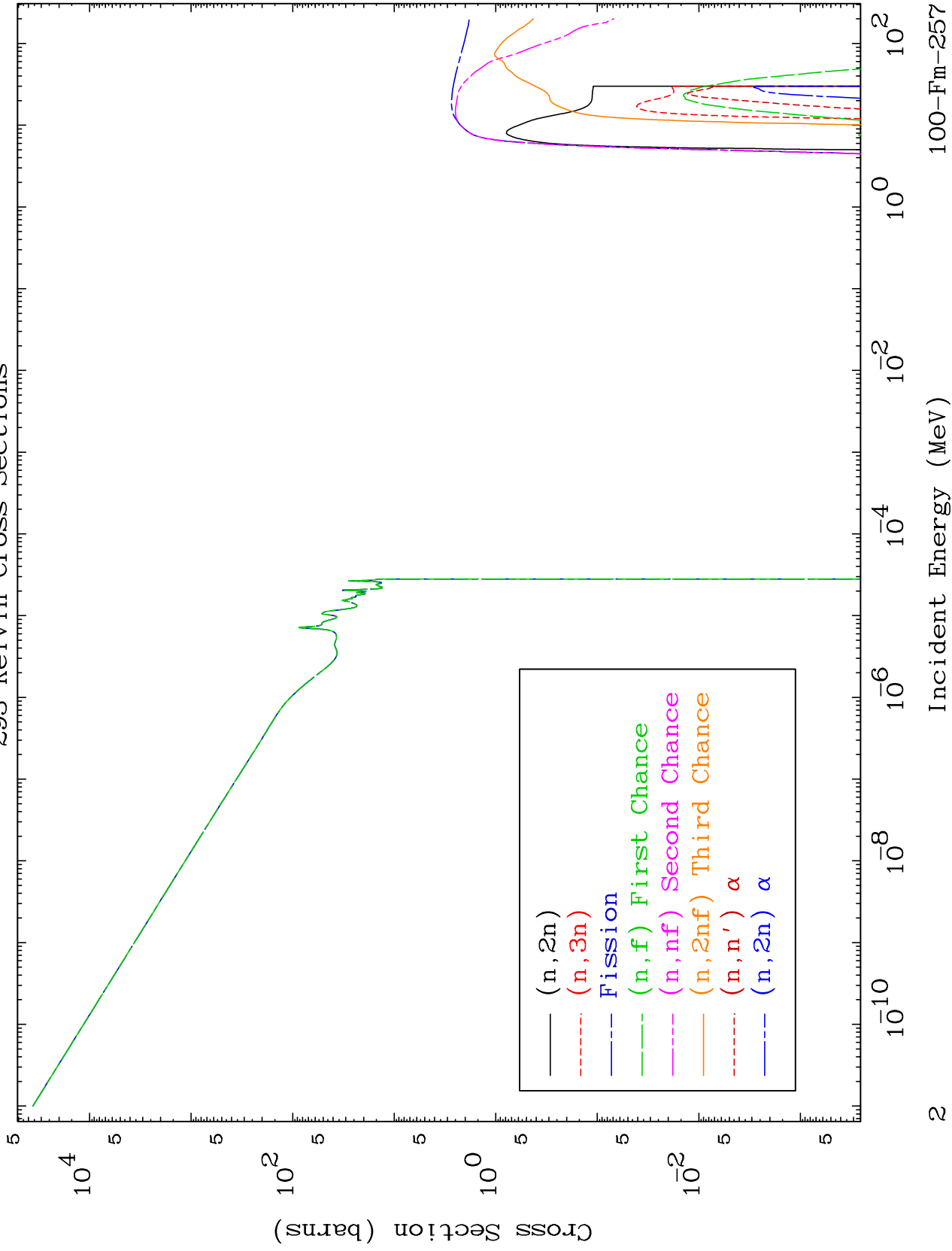
100-Fm-257



MAT 9938

Neutron Absorption
293 Kelvin Cross Sections

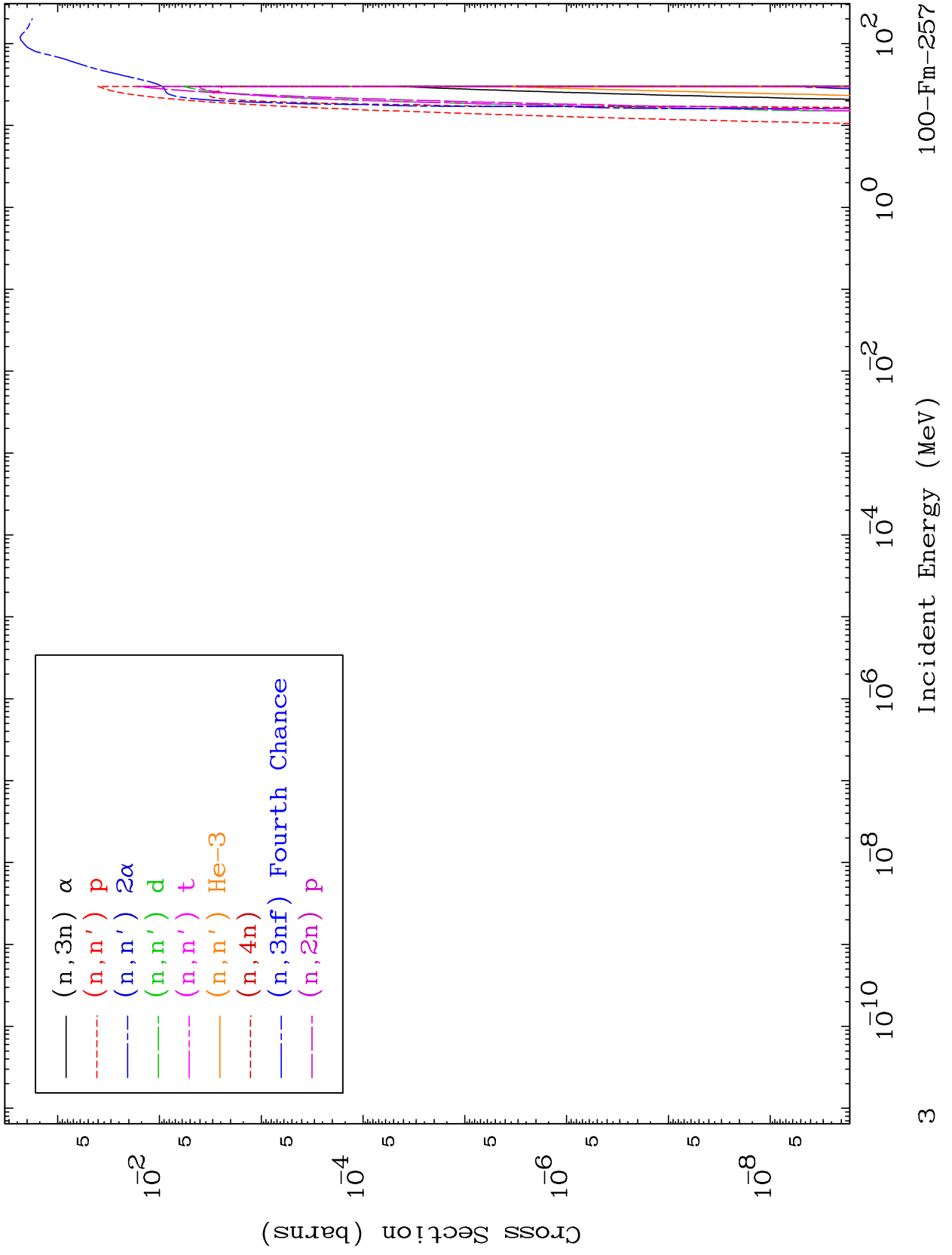
100-Fm-257



MAT 9938

Neutron Absorption
293 Kelvin Cross Sections

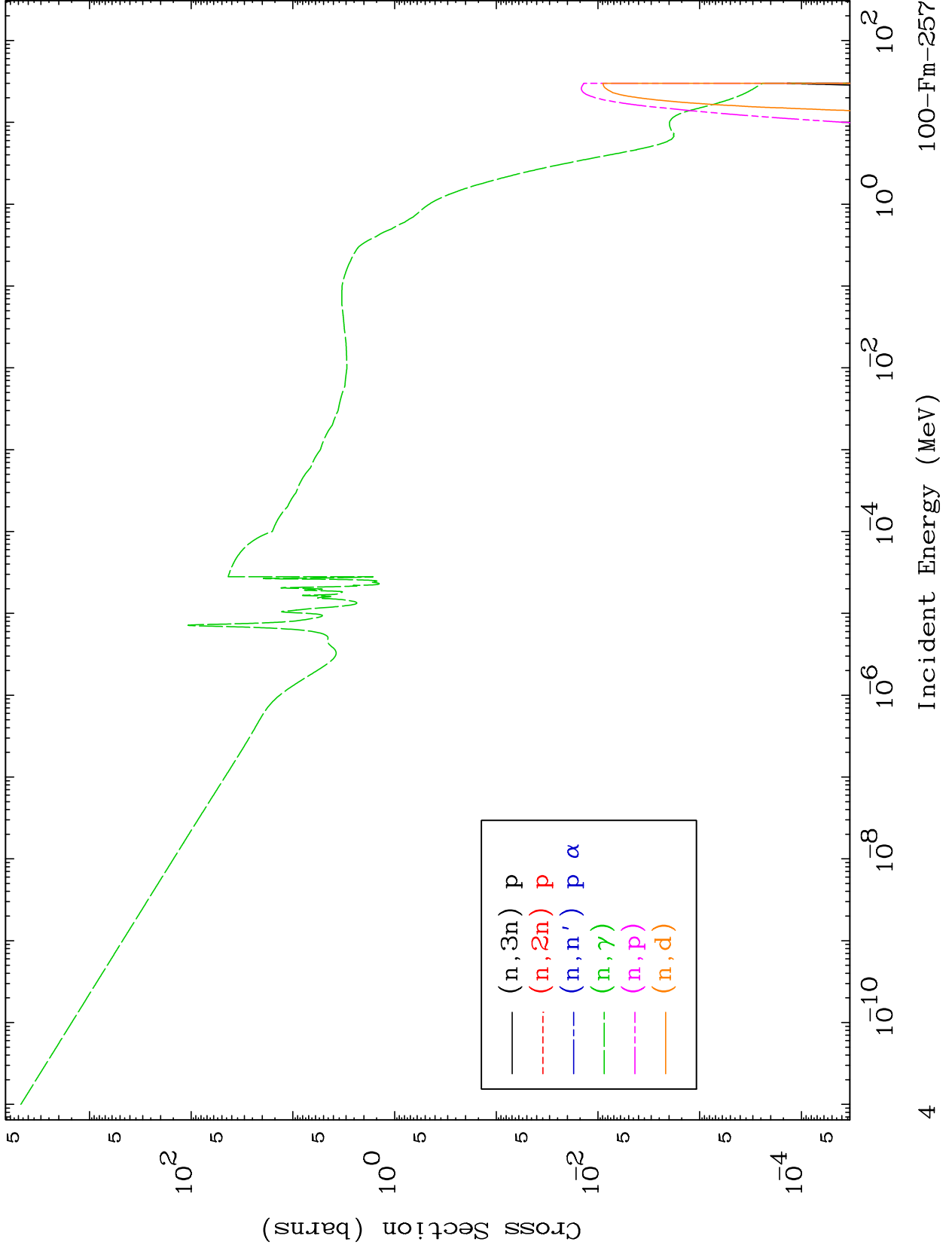
100-Fm-257



MAT 9938

Neutron Absorption
293 Kelvin Cross Sections

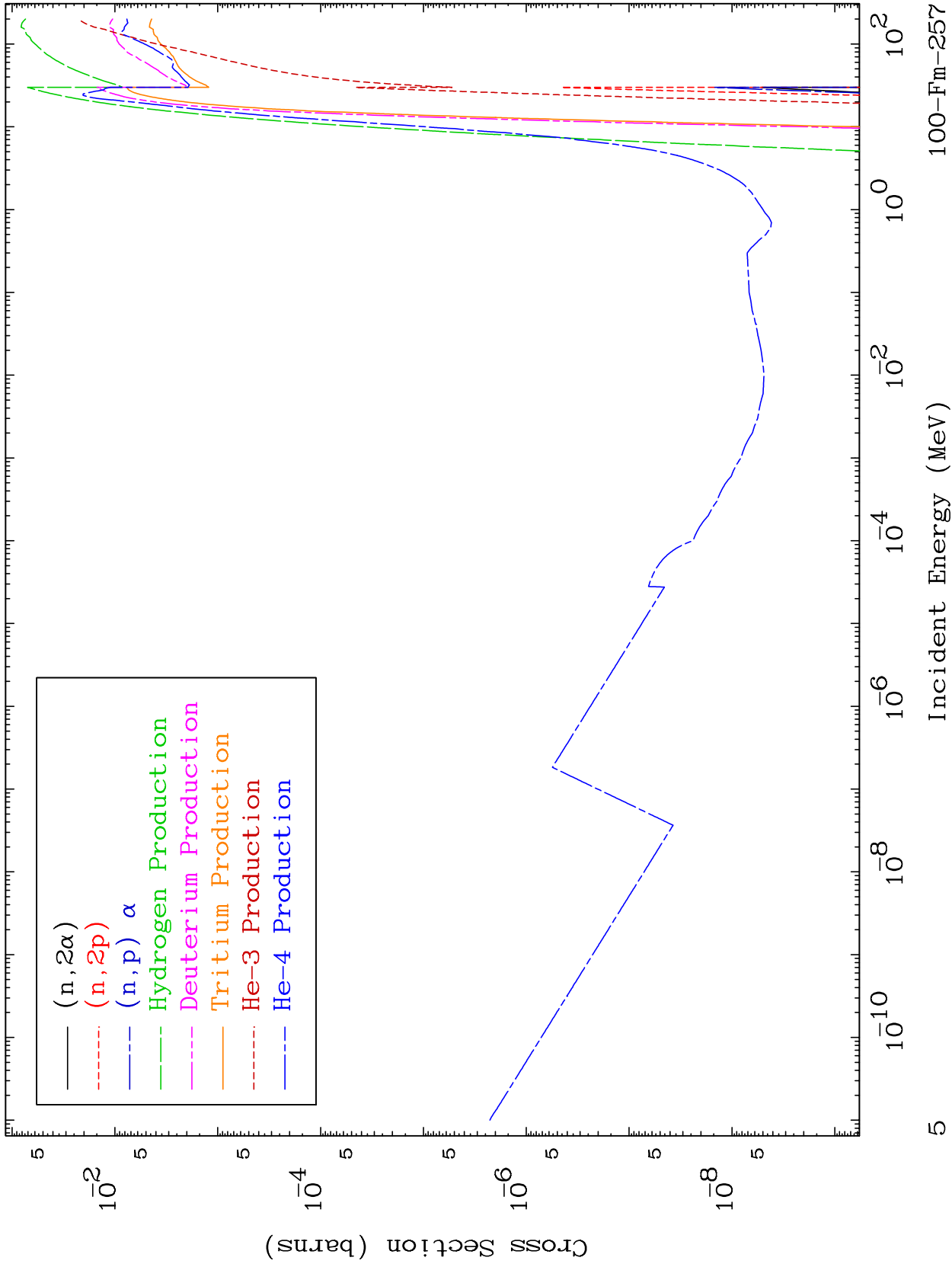
100-Fm-257



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Neutron Absorption
293 Kelvin Cross Sections

100-Fm-257



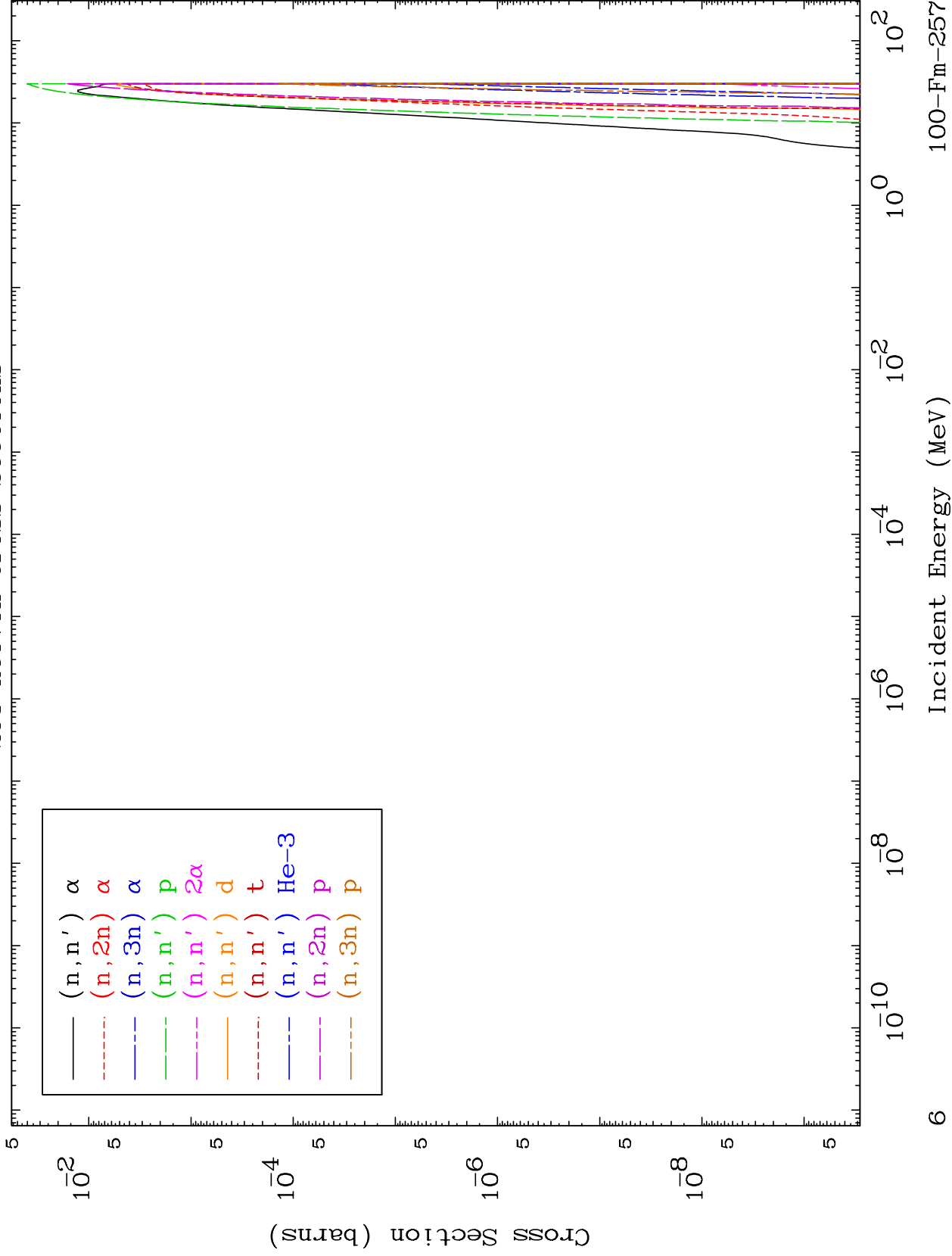
5

100-Fm-257

MAT 9938

Charged Particle
293 Kelvin Cross Sections

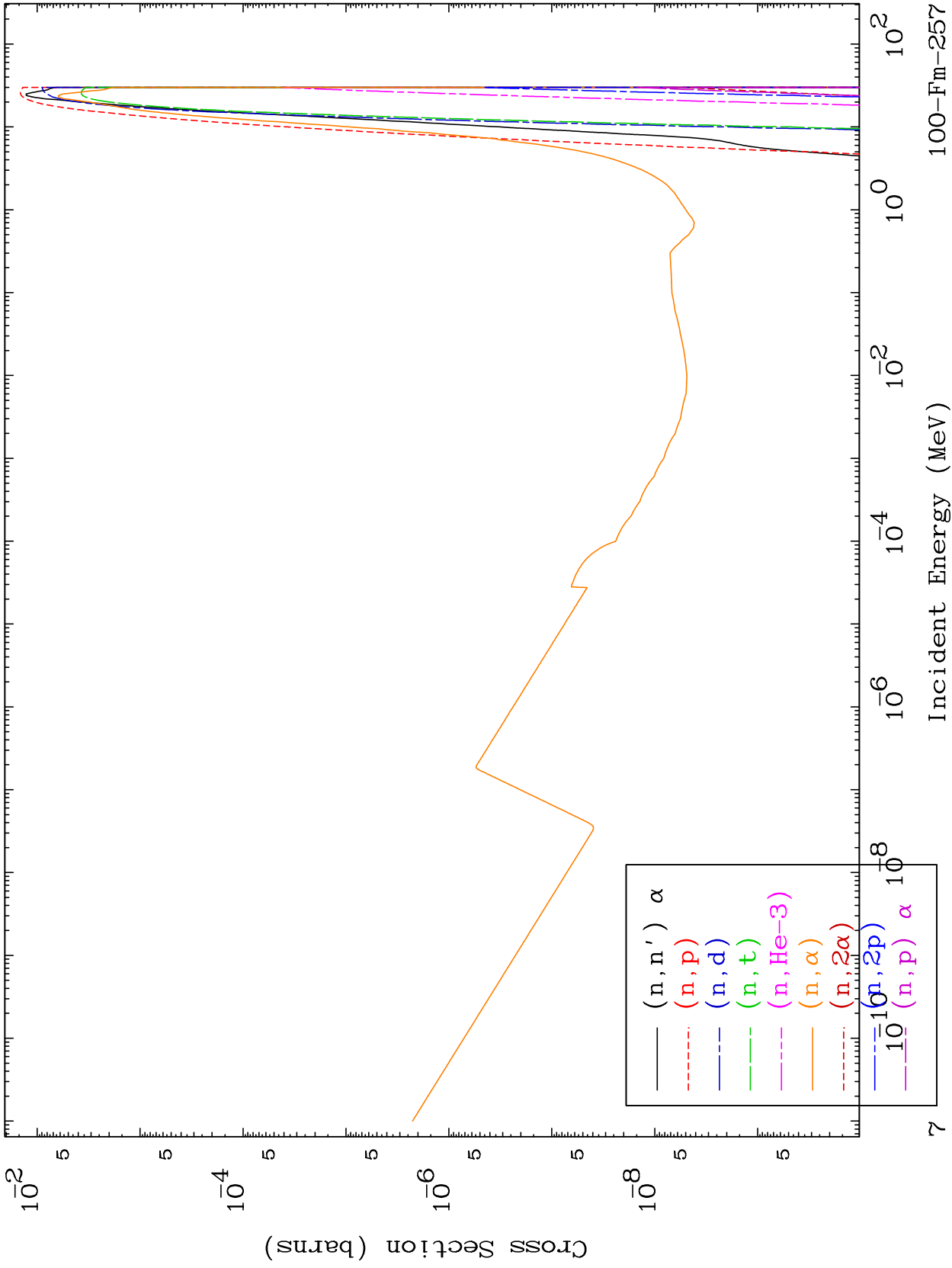
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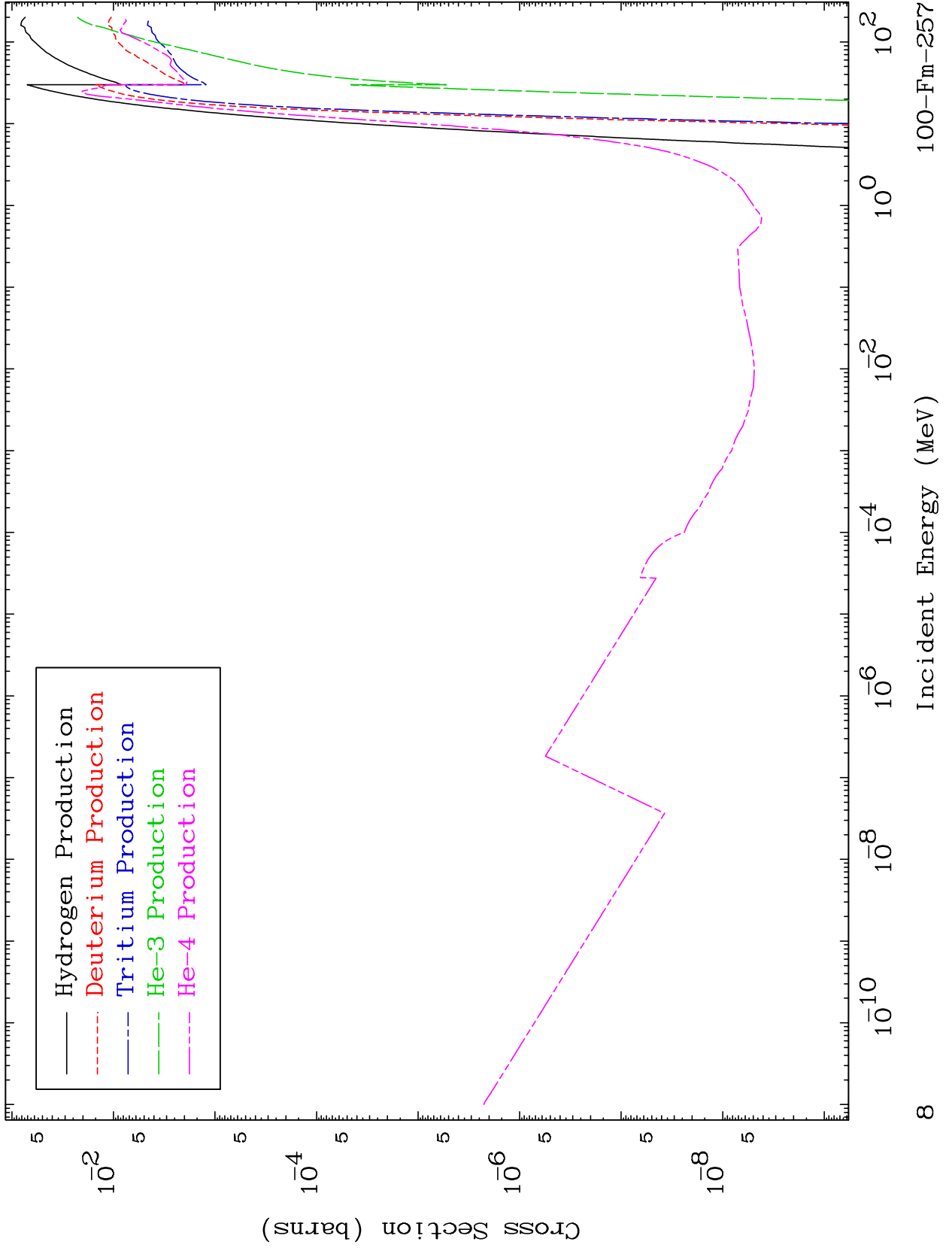


MAT 9938

Charged Particle
293 Kelvin Cross Sections

100-Fm-257

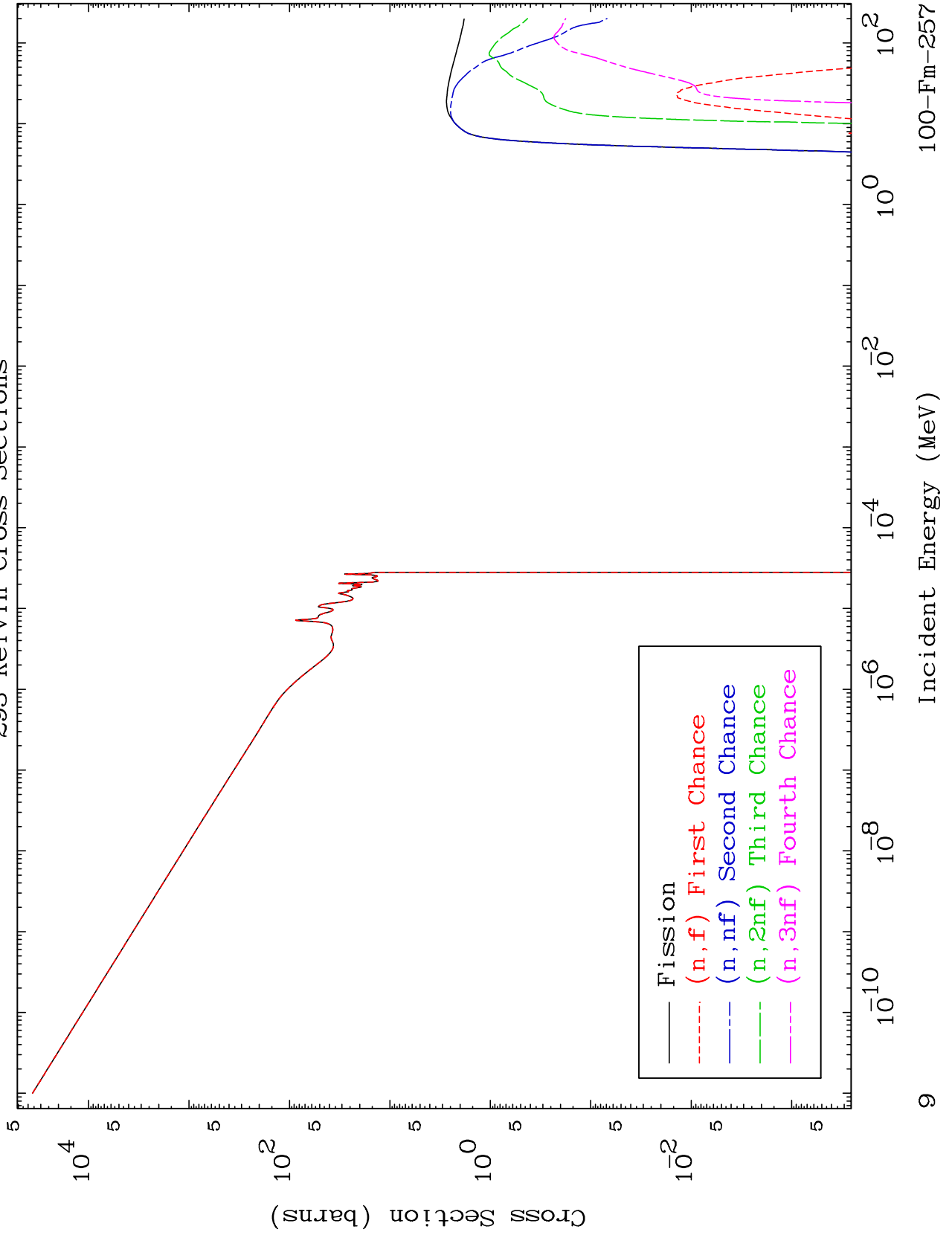




MAT 9938

Fission
293 Kelvin Cross Sections

100-Fm-257

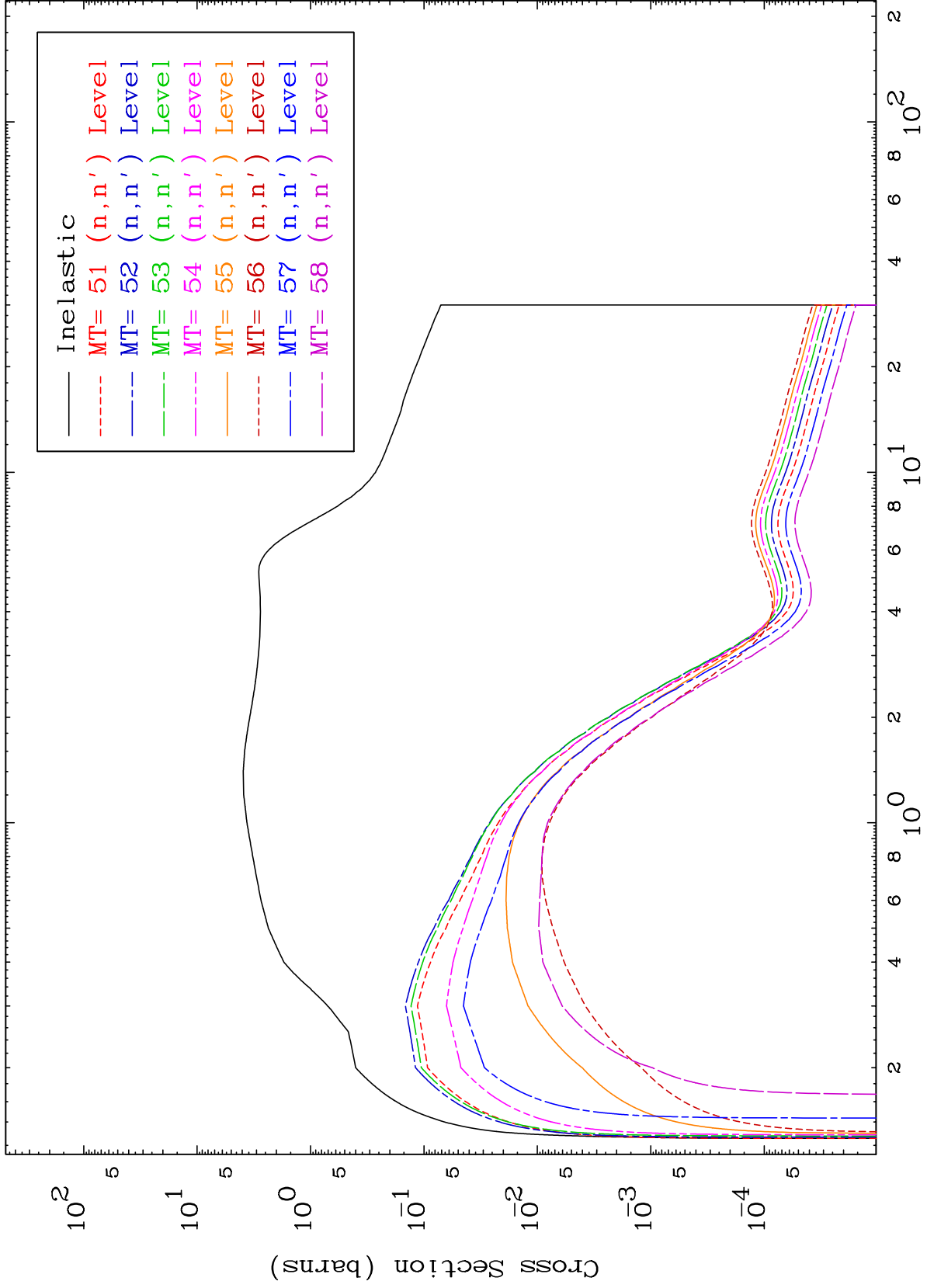


MAT 9938

(n,n') Levels

100-Fm-257

293 Kelvin Cross Sections



10

Incident Energy (MeV)

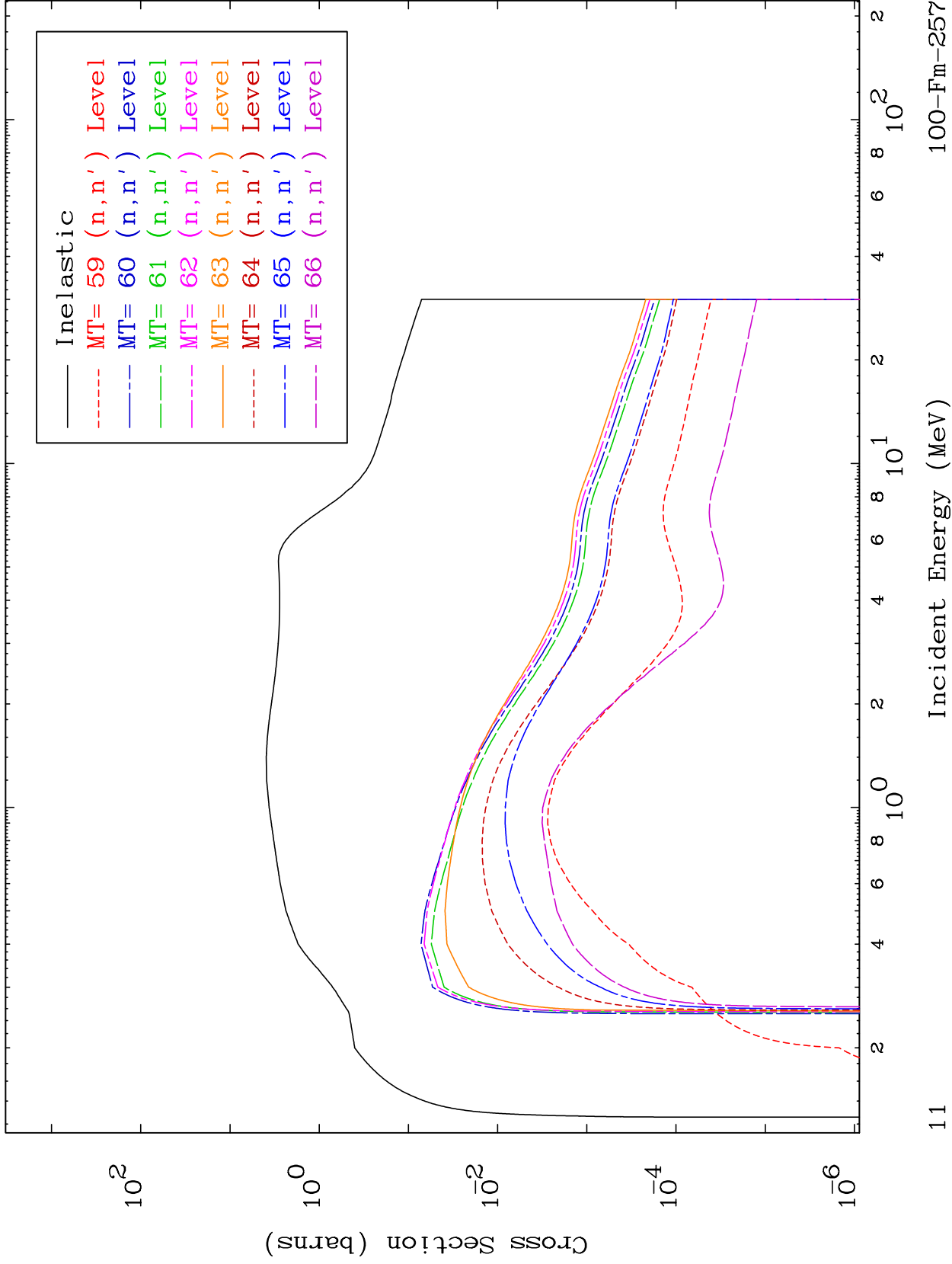
100-Fm-257

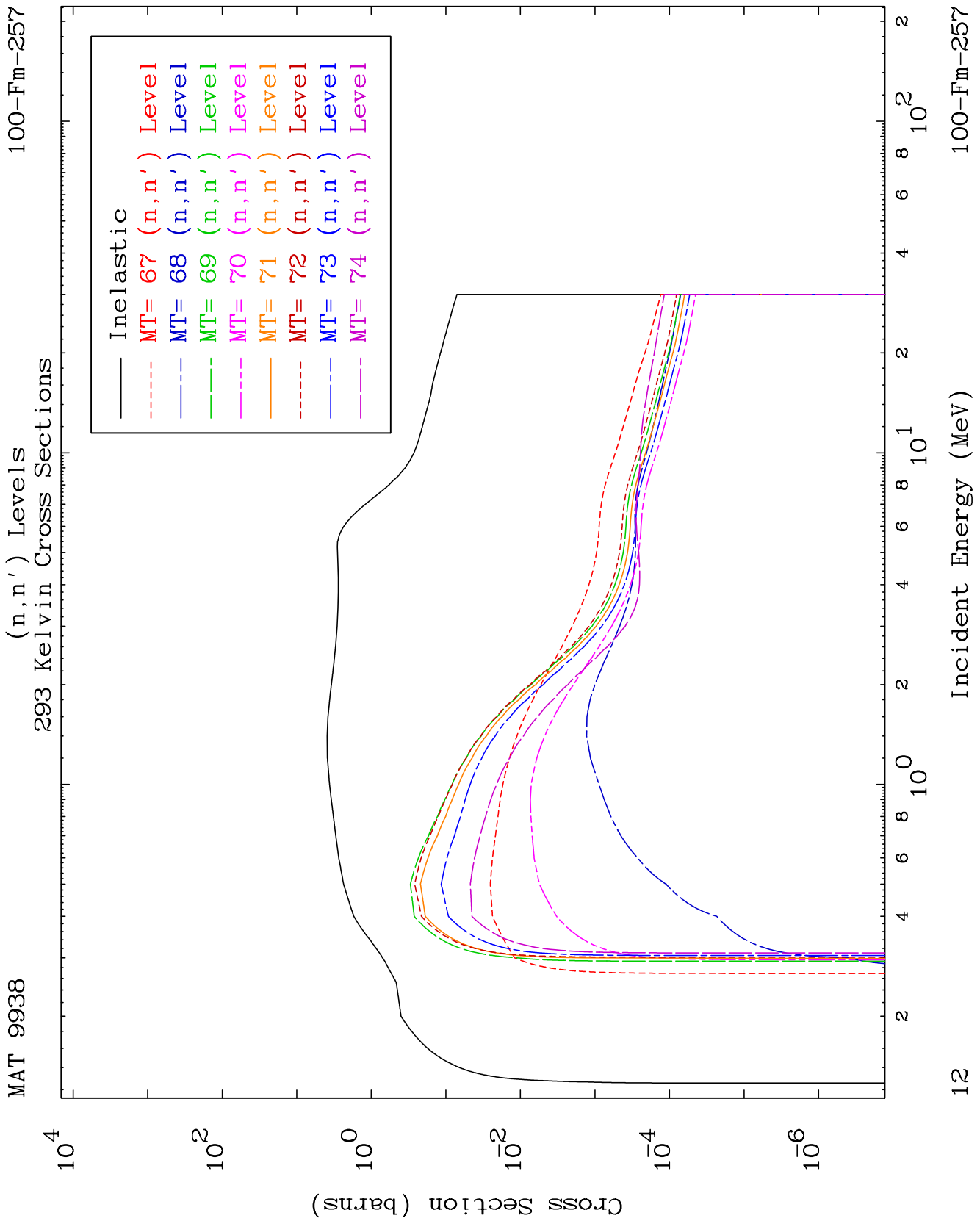
MAT 9938

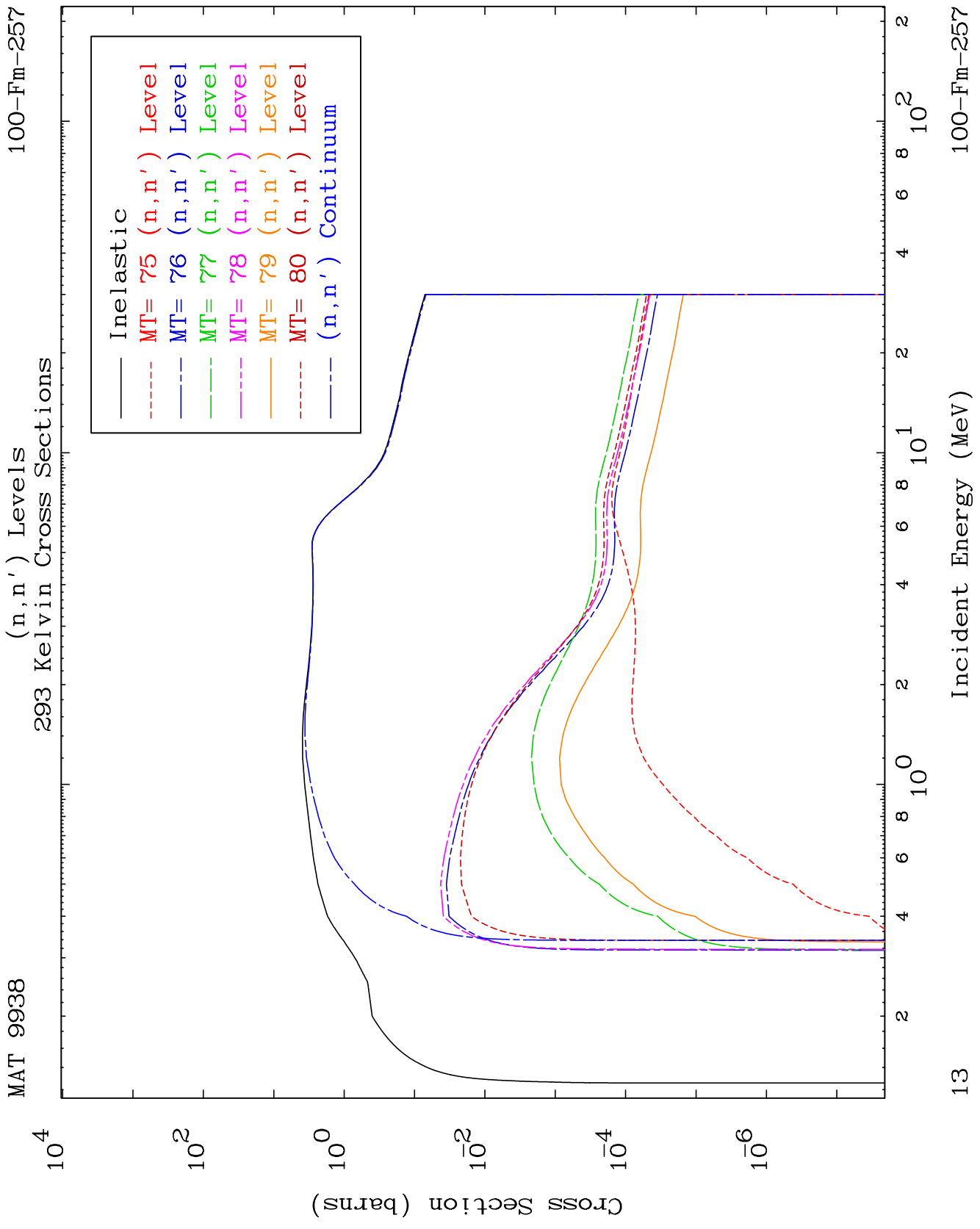
(n,n') Levels

100-Fm-257

293 Kelvin Cross Sections



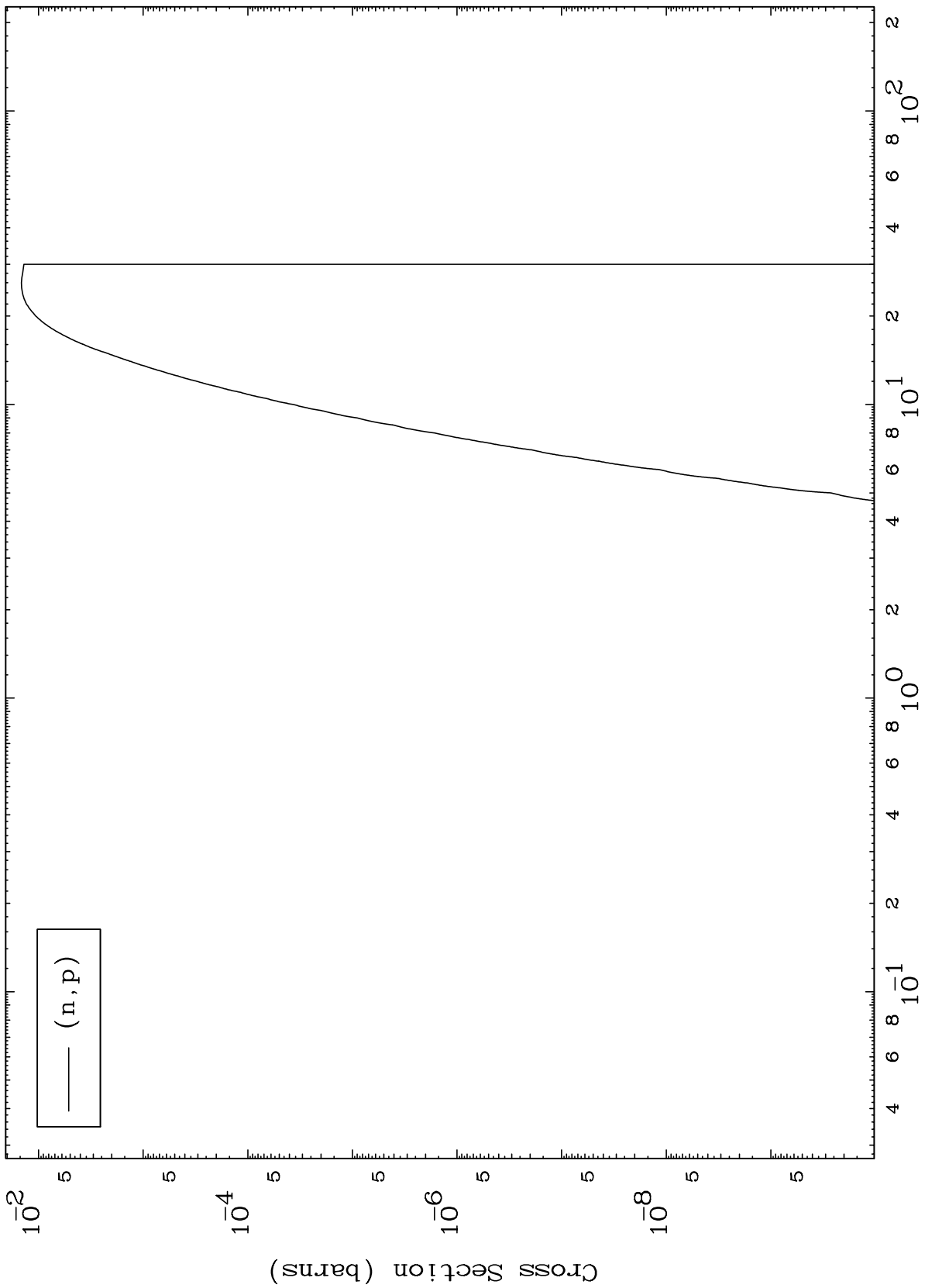




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(n,p) Levels
293 Kelvin Cross Sections

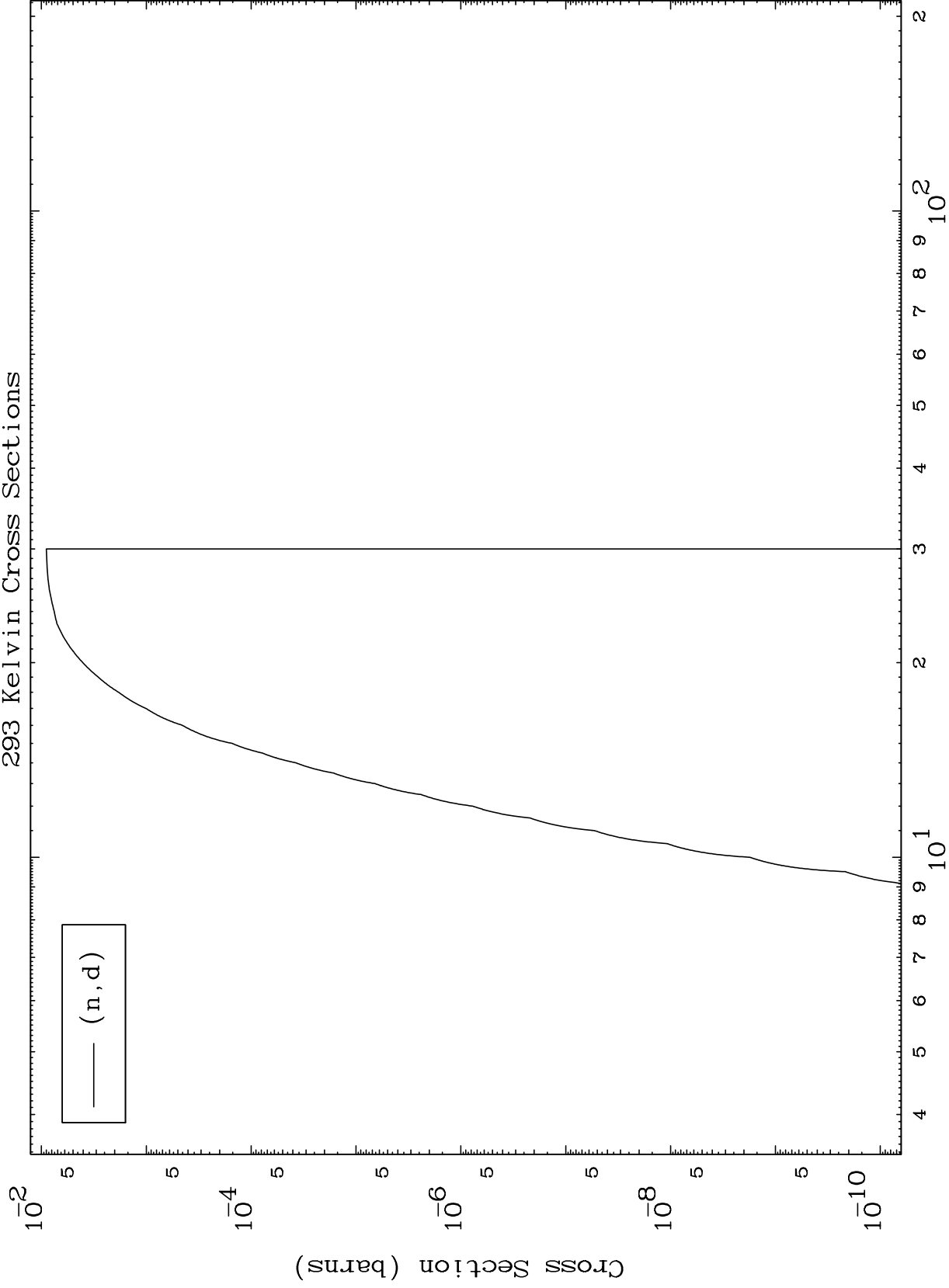
100-Fm-257



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(n,d) Levels
293 Kelvin Cross Sections

100-Fm-257



15

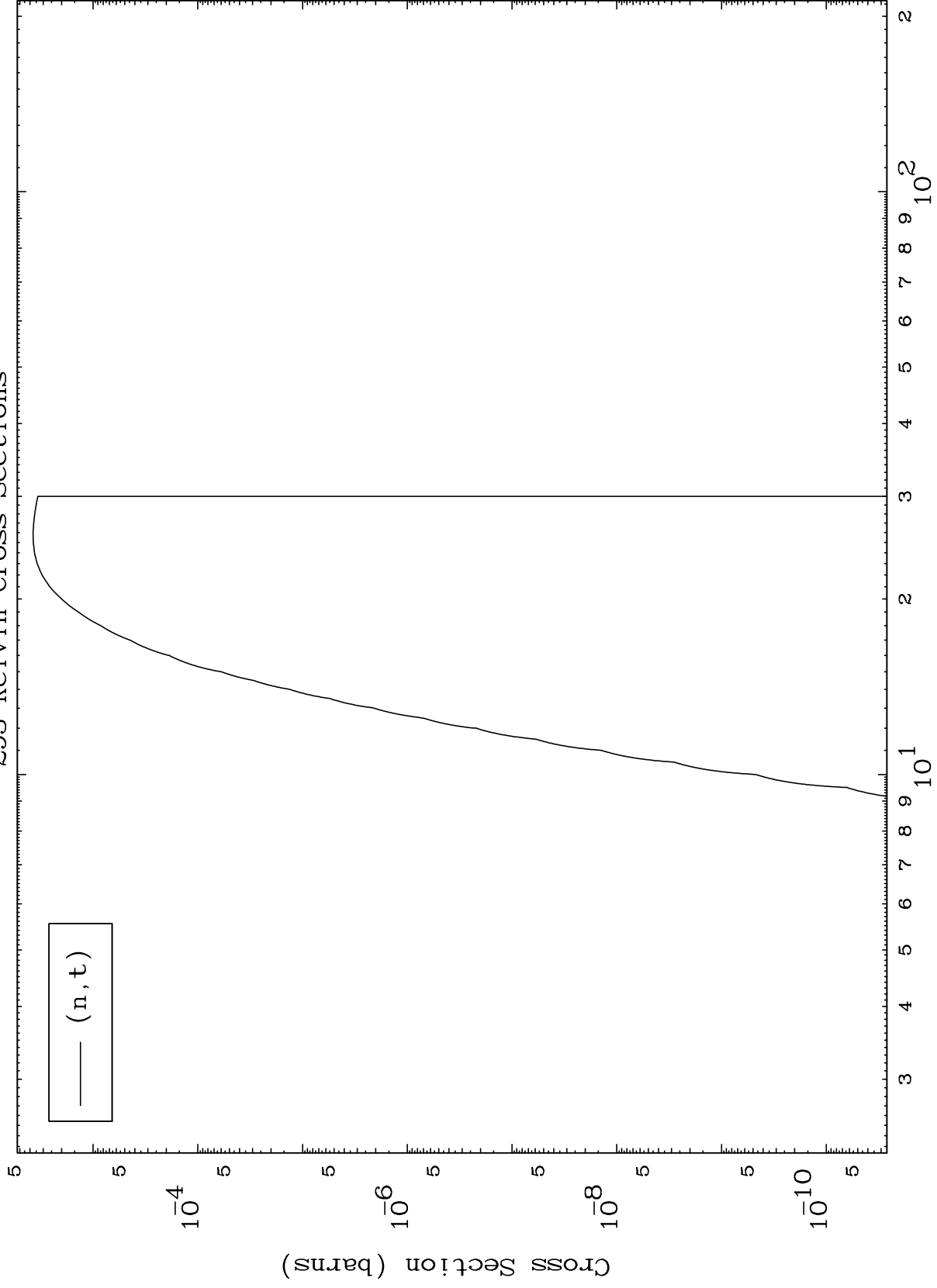
Incident Energy (MeV)

100-Fm-257

MAT 9938

(n,t) Levels
293 Kelvin Cross Sections

100-Fm-257



16

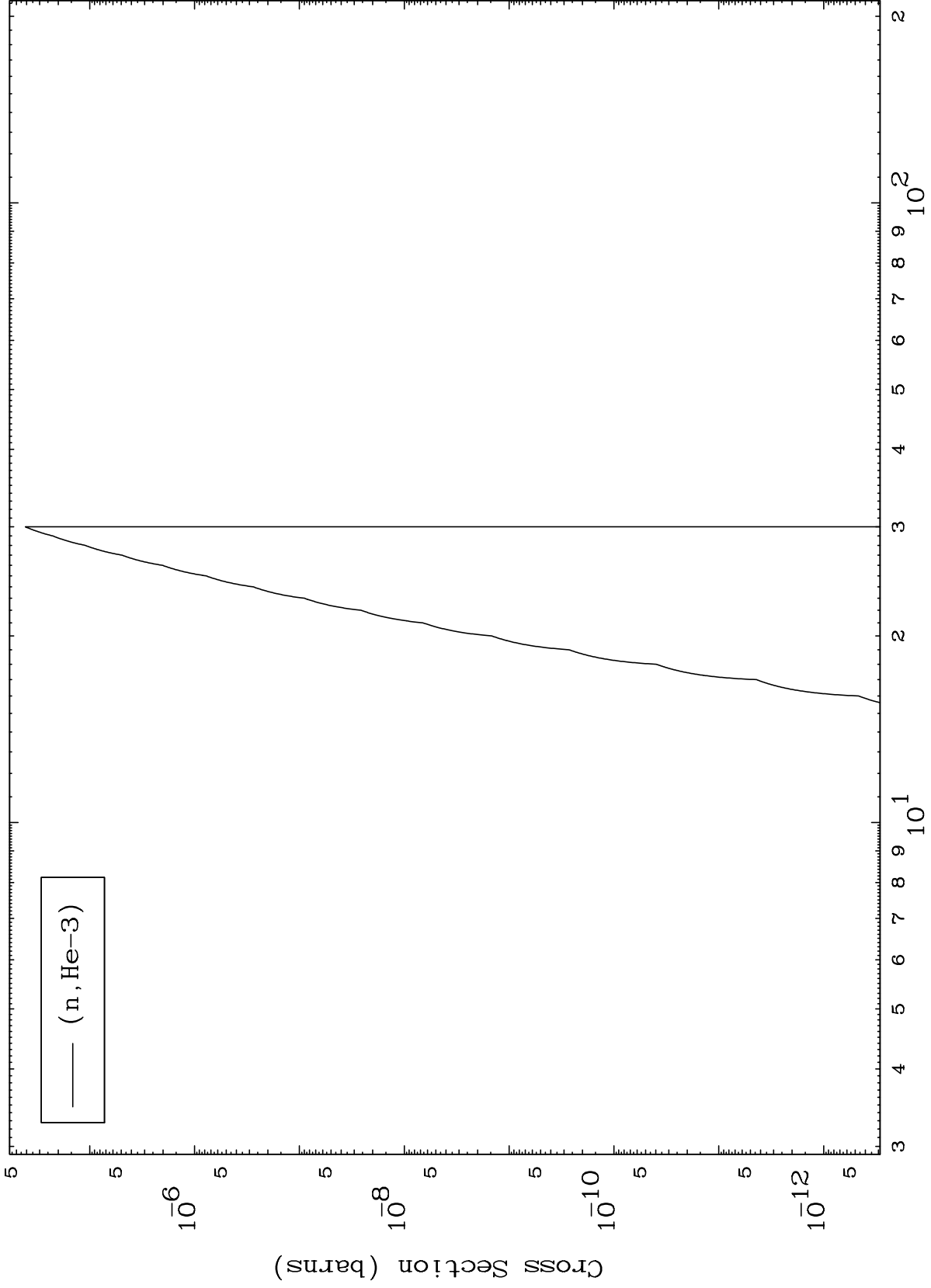
Incident Energy (MeV)

100-Fm-257

MAT 9938

(n,He3) Levels
293 Kelvin Cross Sections

100-Fm-257



17

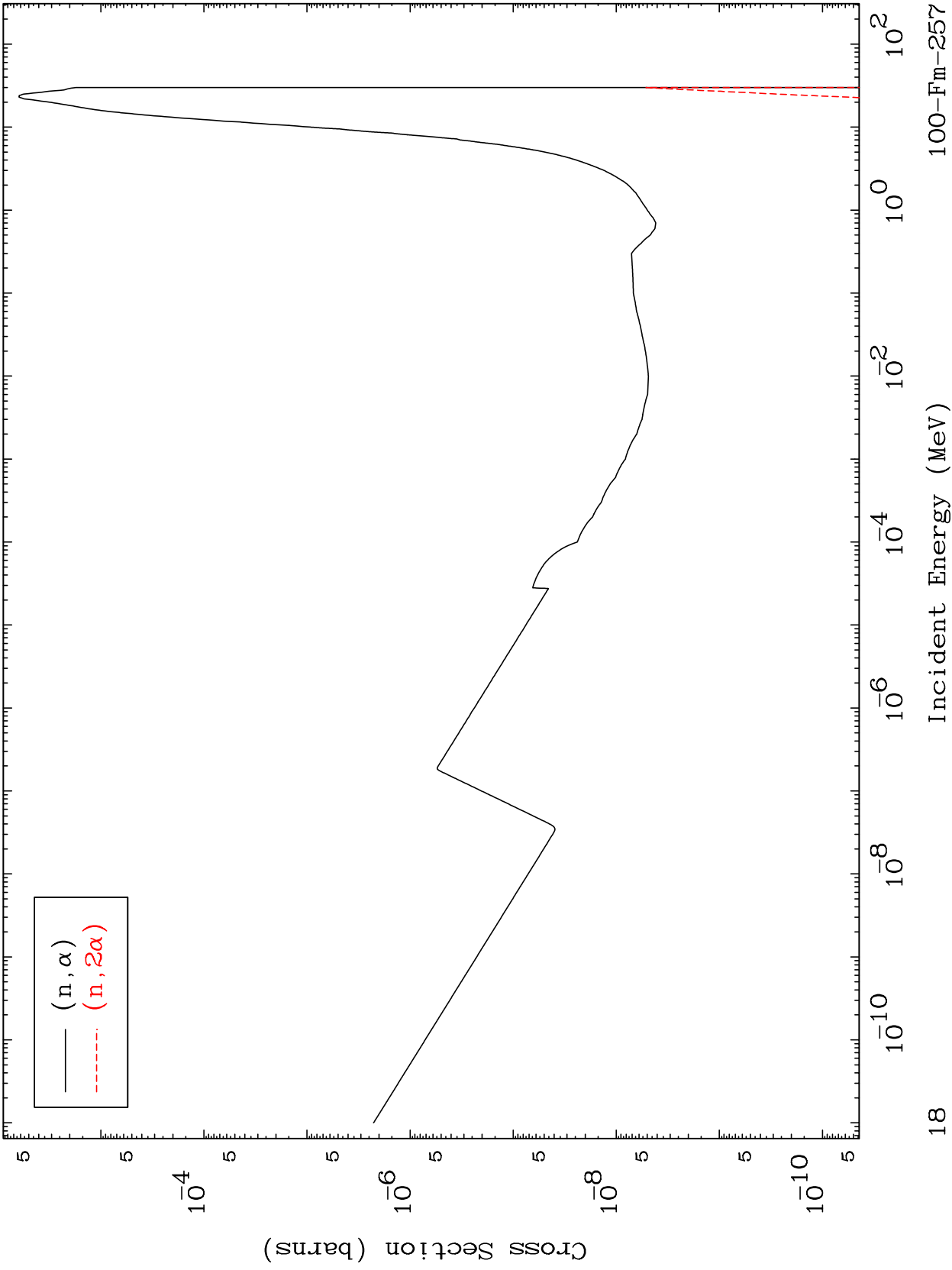
Incident Energy (MeV)

100-Fm-257

MAT 9938

(n, α) Levels
293 Kelvin Cross Sections

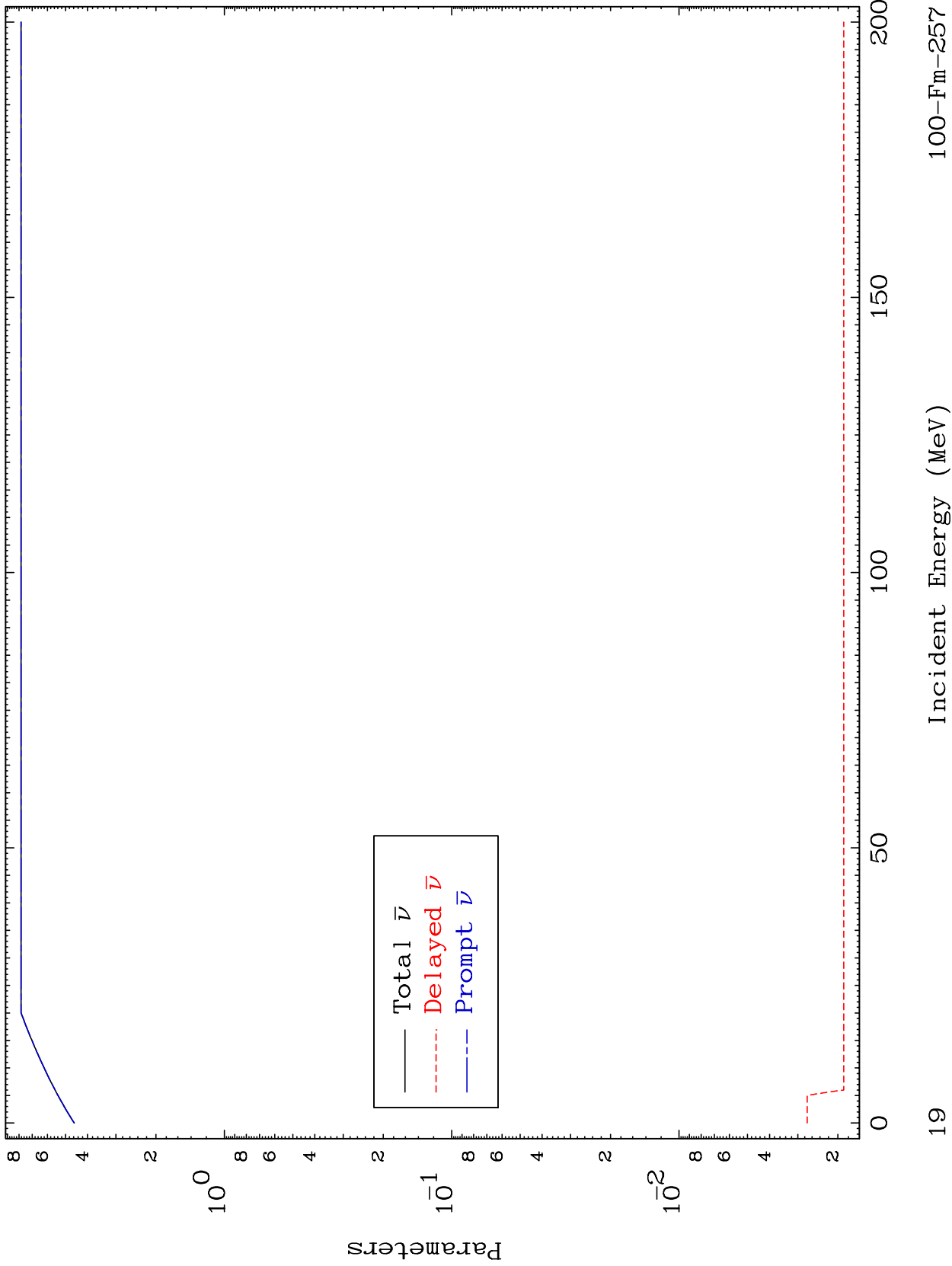
100-Fm-257



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Energy Release
Parameters

100-Fm-257



19

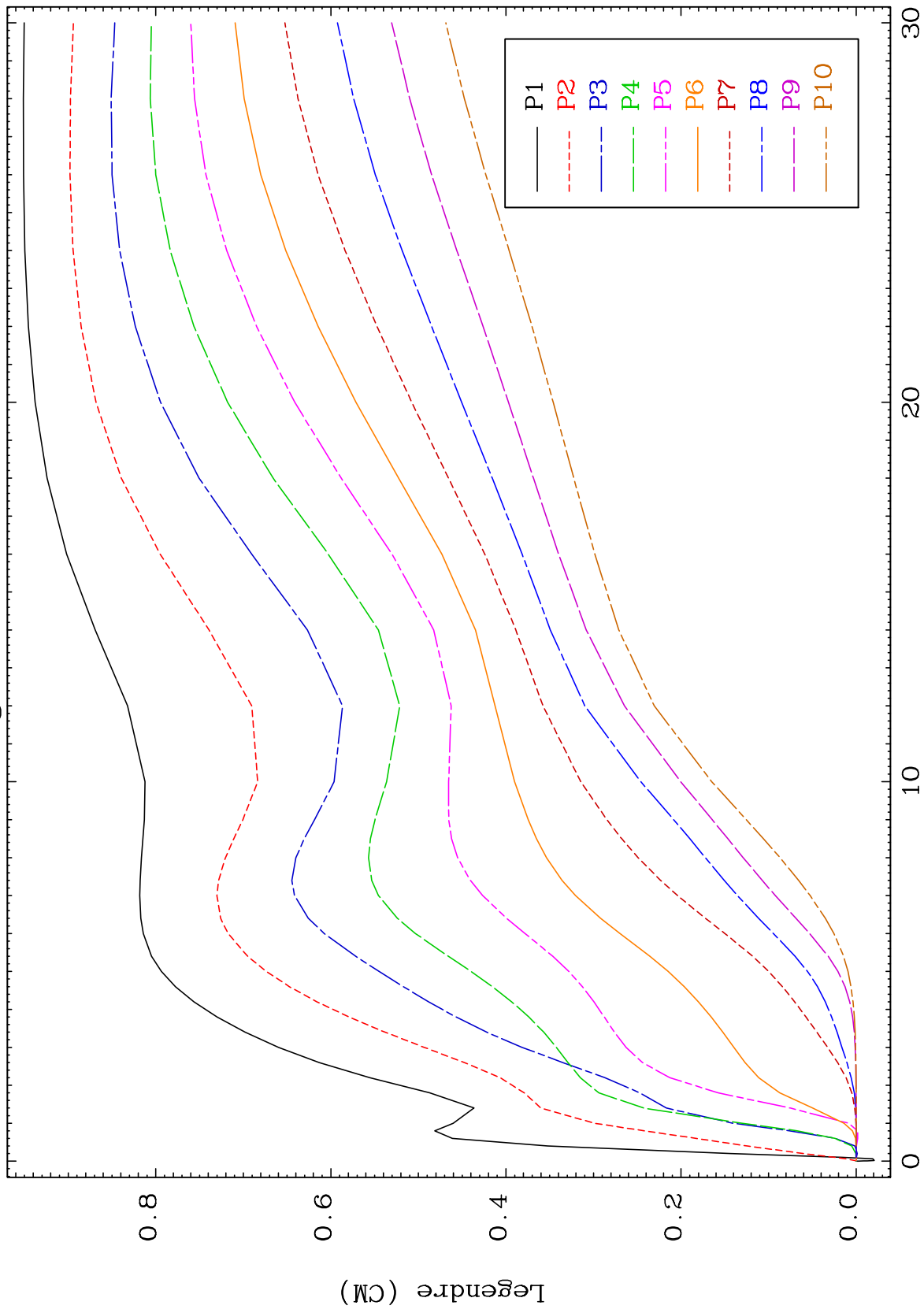
Incident Energy (MeV)

100-Fm-257

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Elastic Legendre Coefficients

100-Fm-257



0

10

Incident Energy (MeV)

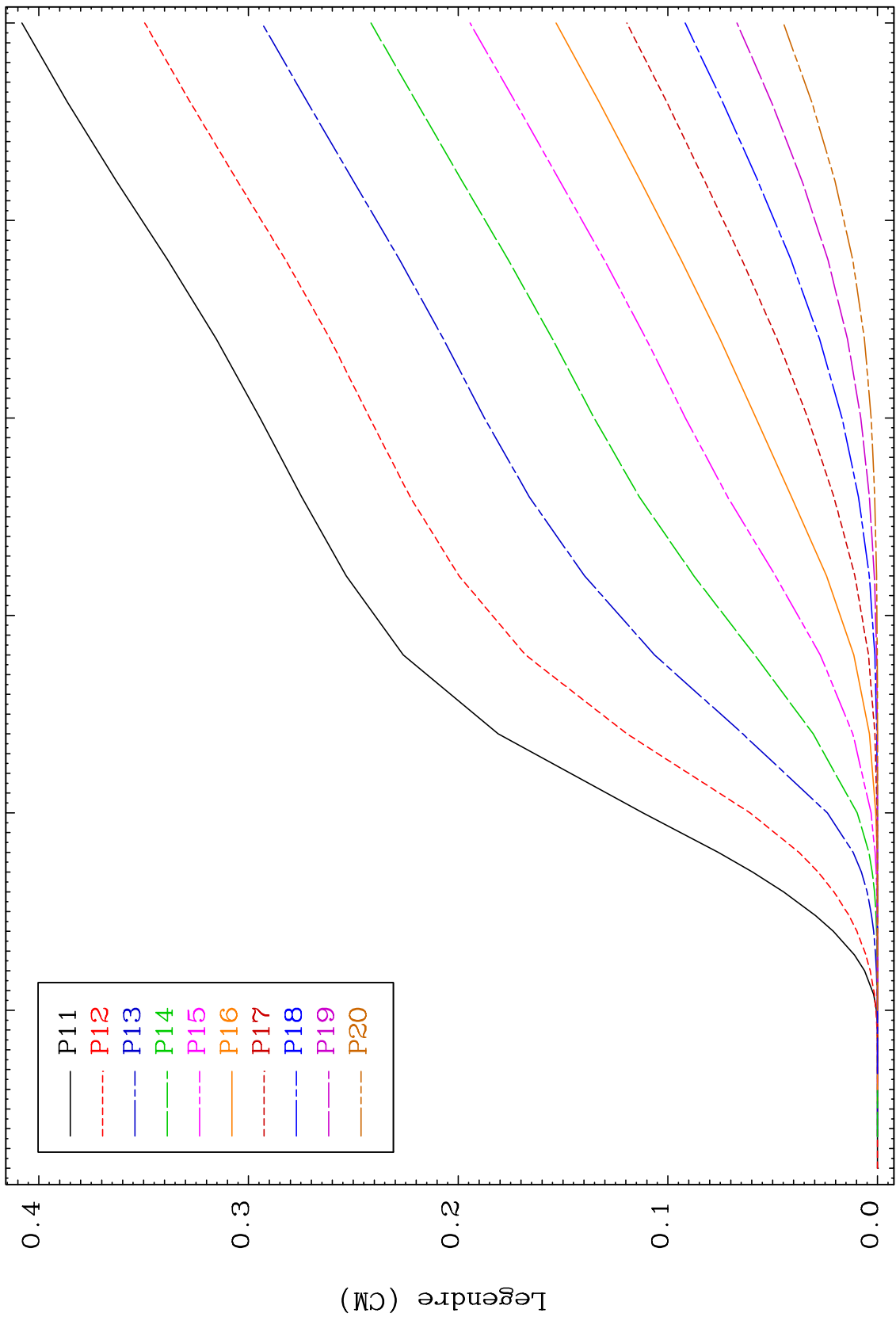
100-Fm-257

30

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Elastic Legendre Coefficients

100-Fm-257



21

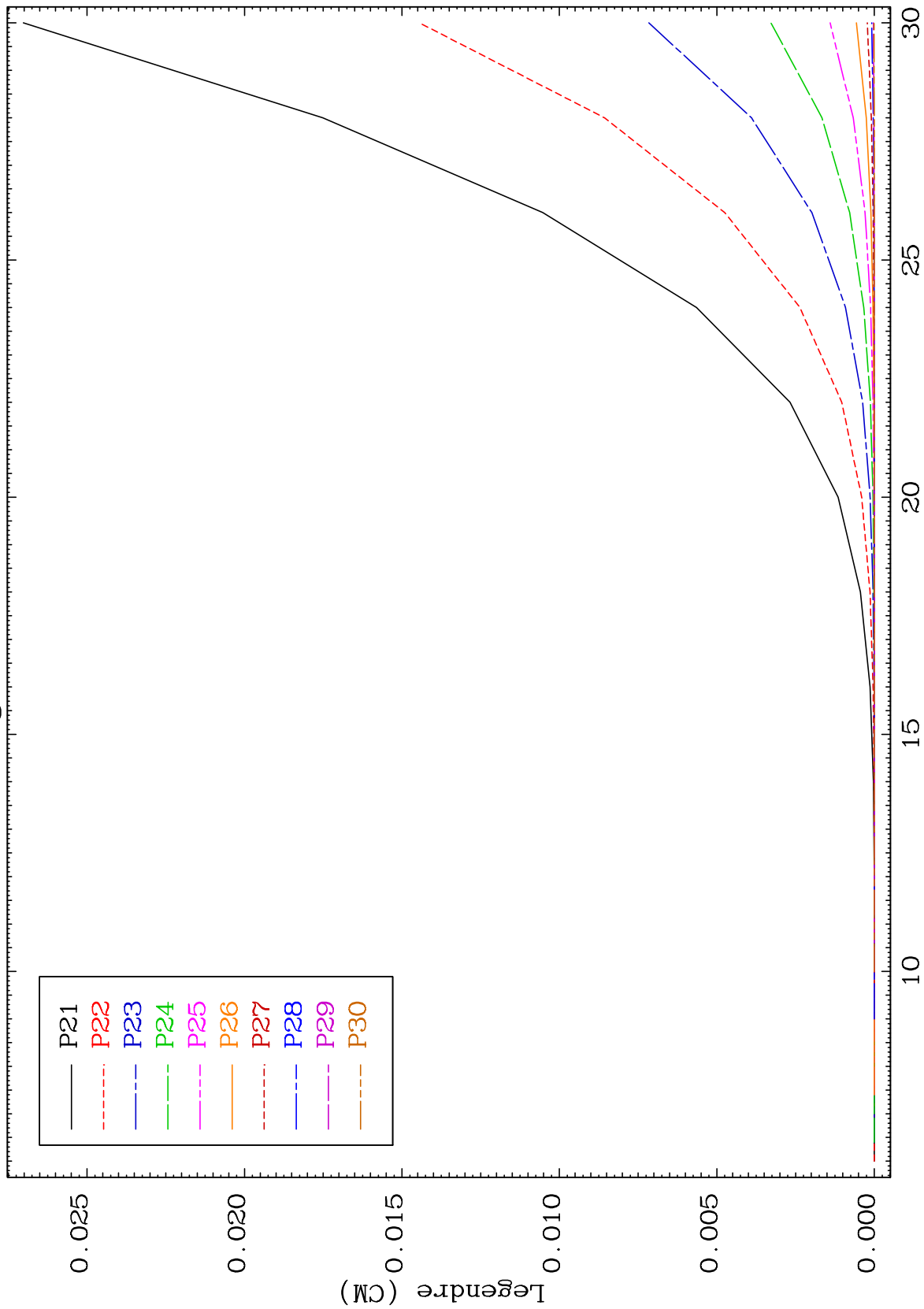
Incident Energy (MeV)

100-Fm-257

MAT 9938

Elastic Legendre Coefficients

100-Fm-257



22

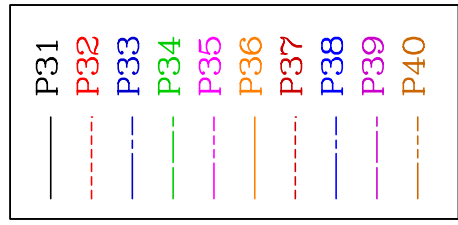
Incident Energy (MeV)

100-Fm-257

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Elastic Legendre Coefficients

100-Fm-257



$\times 10^{-6}$

3

Legendre (CM)

1

0

15

20

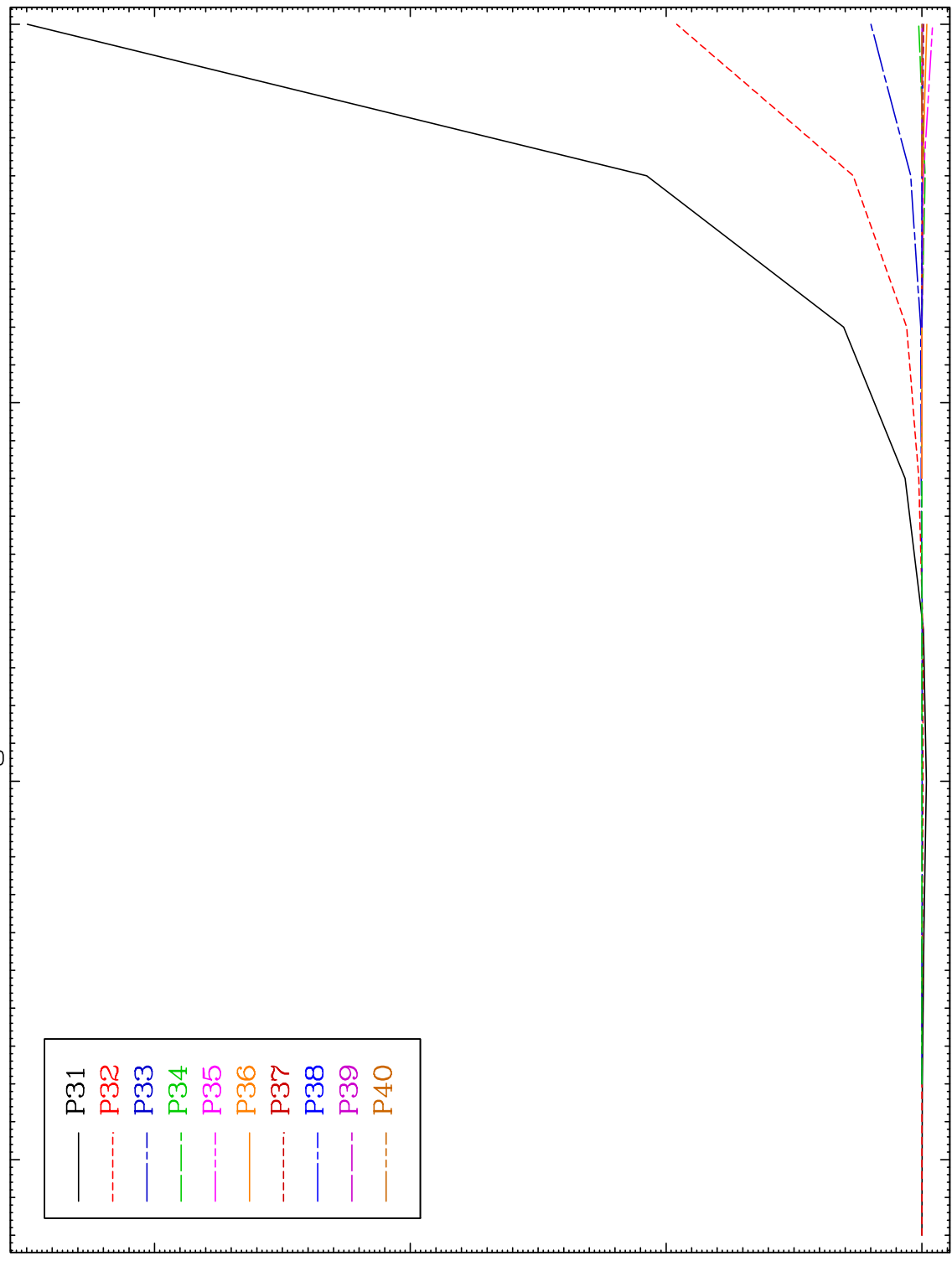
25

30

23

Incident Energy (MeV)

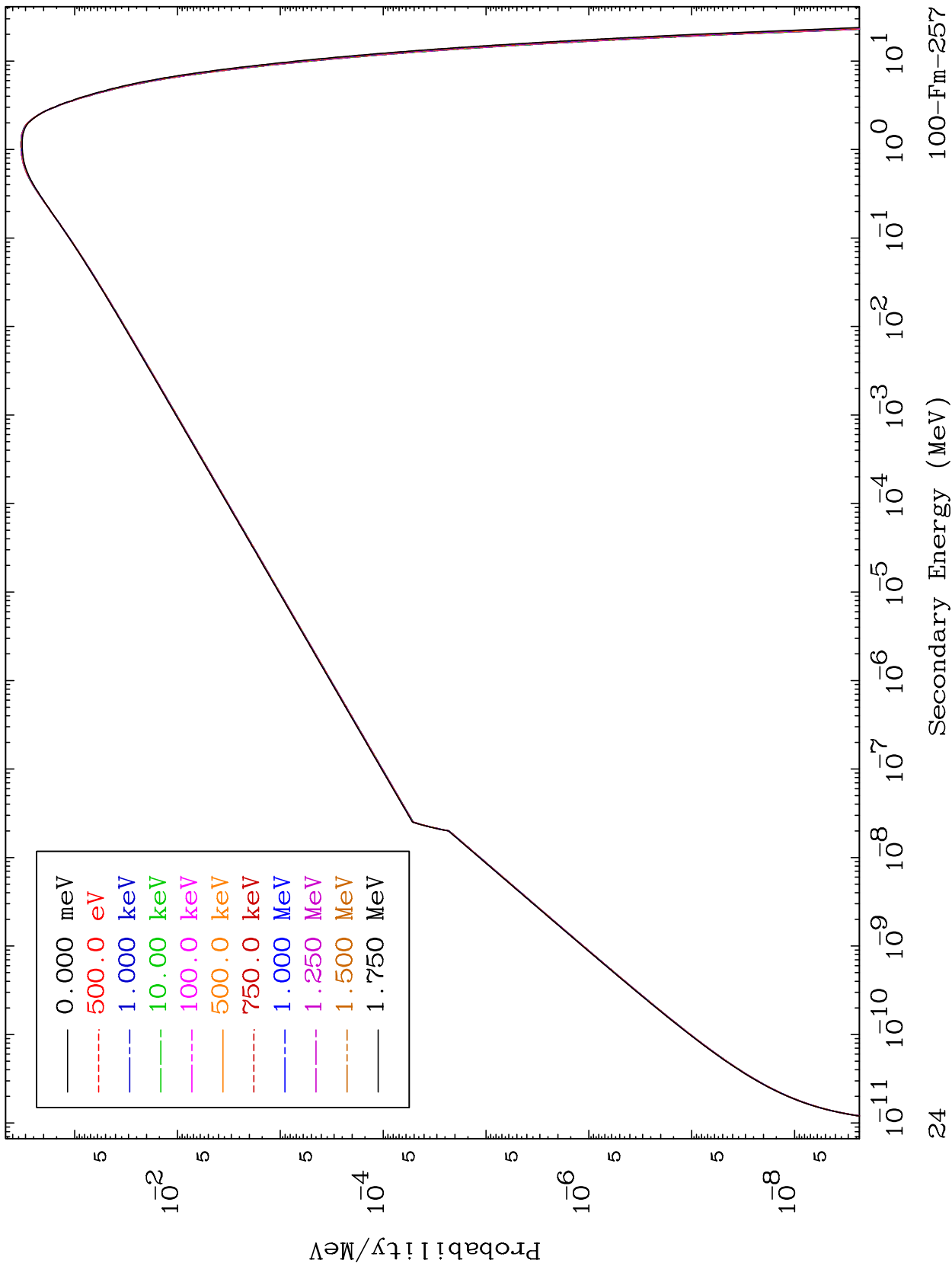
100-Fm-257



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Fission Energy Distributions

100-Fm-257

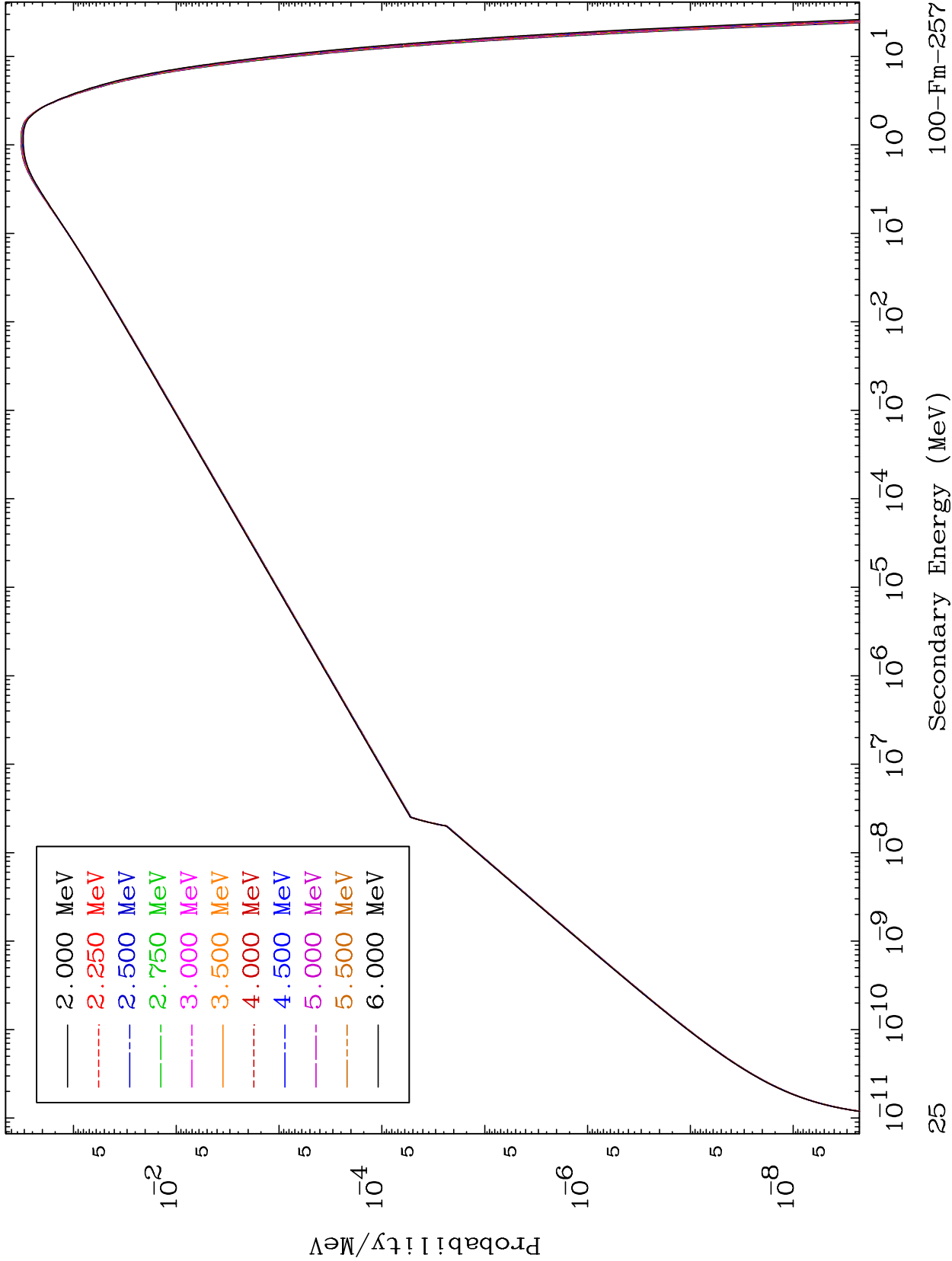


100-Fm-257

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Fission Energy Distributions

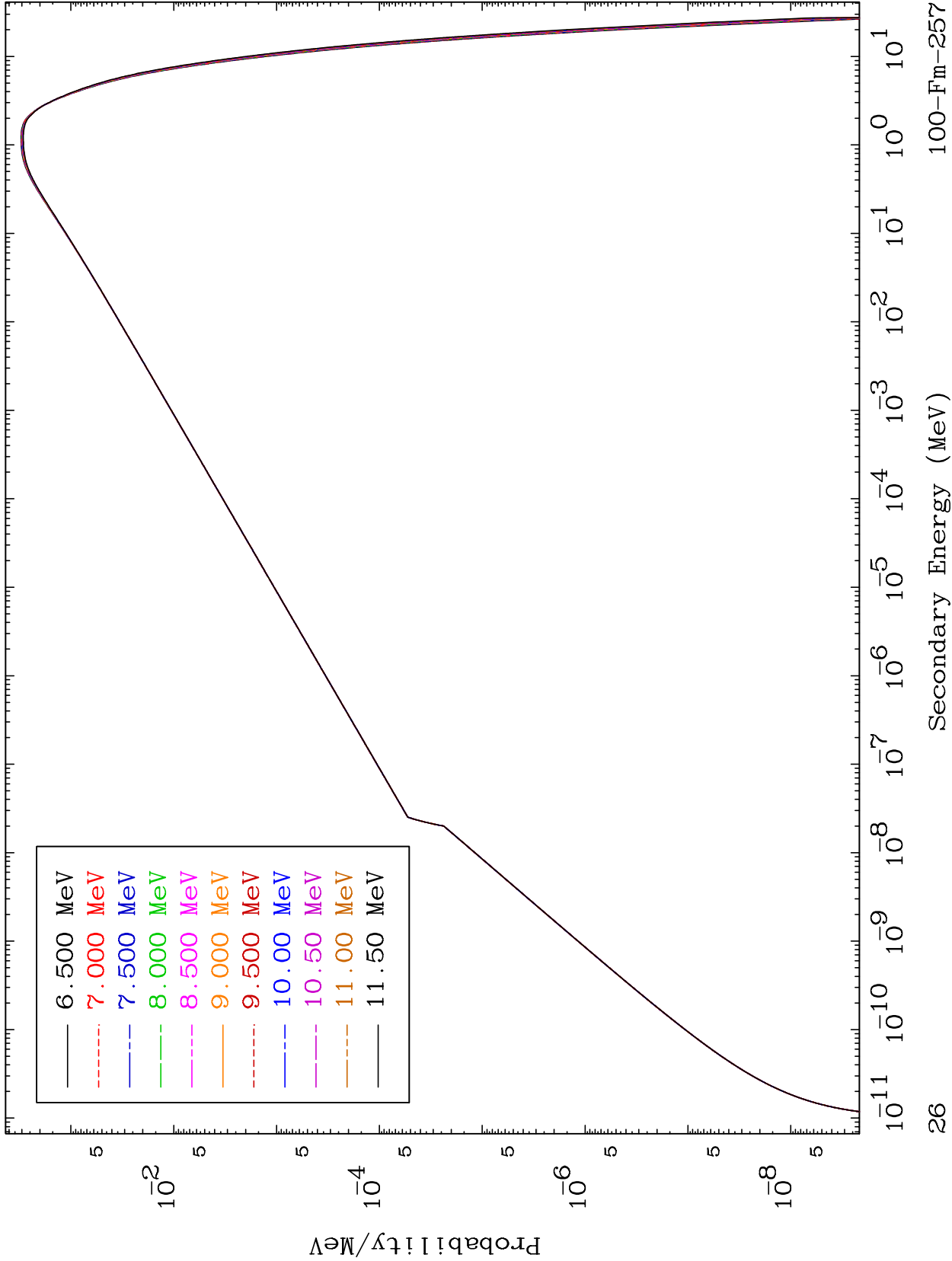
100-Fm-257



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Fission Energy Distributions

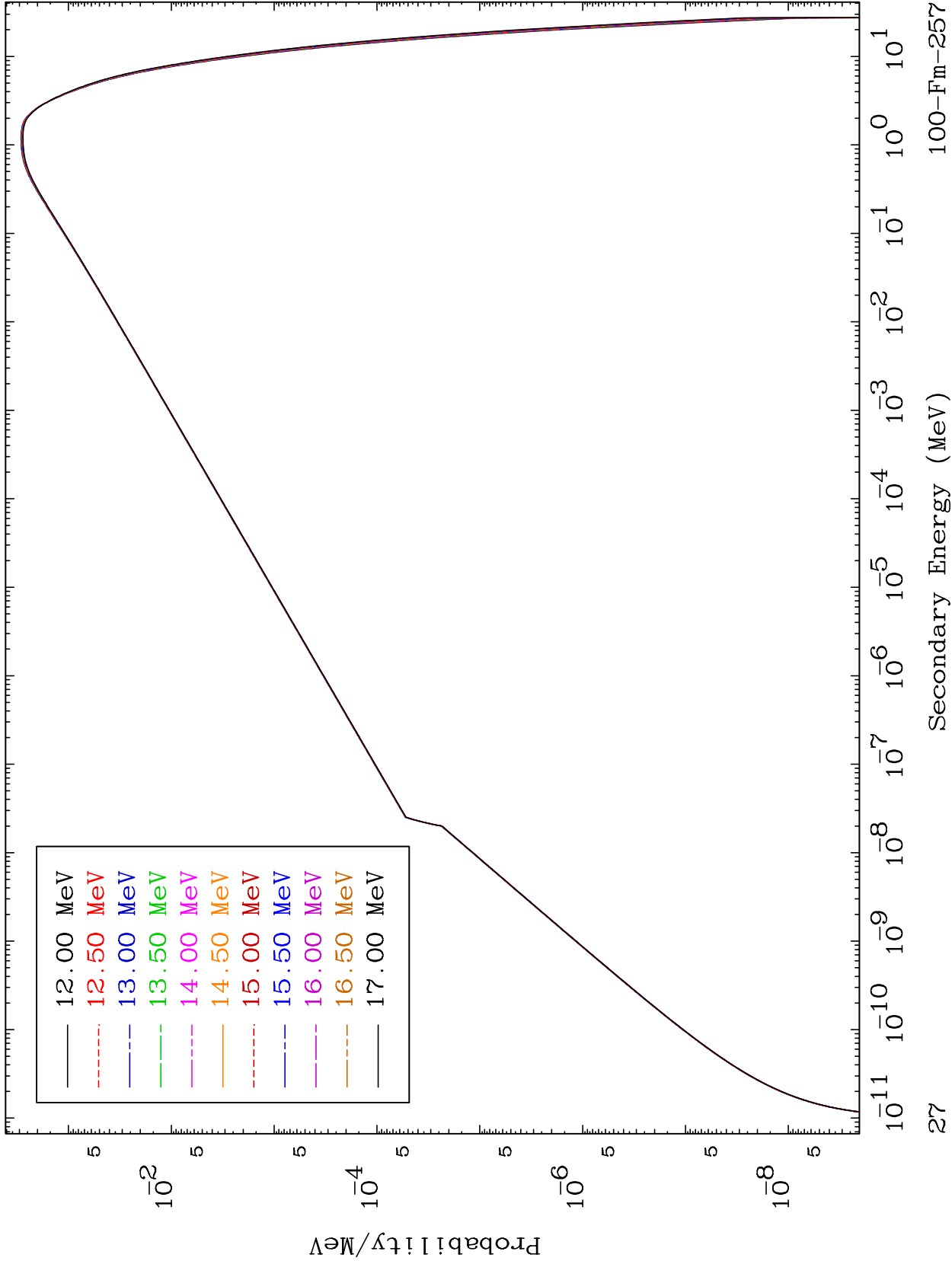
100-Fm-257



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Fission Energy Distributions

100-Fm-257



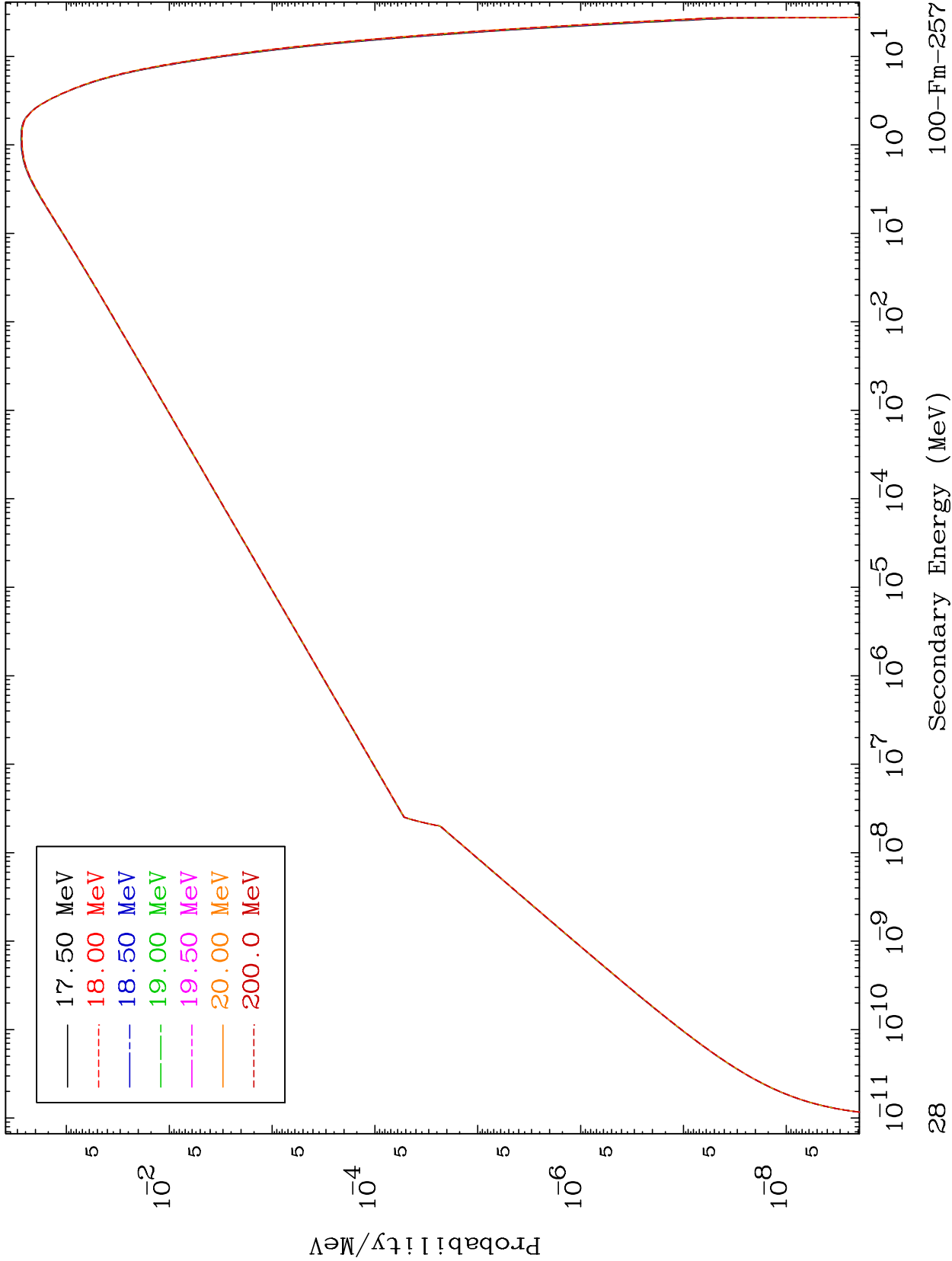
100-Fm-257

Secondary Energy (MeV)

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Fission Energy Distributions

100-Fm-257

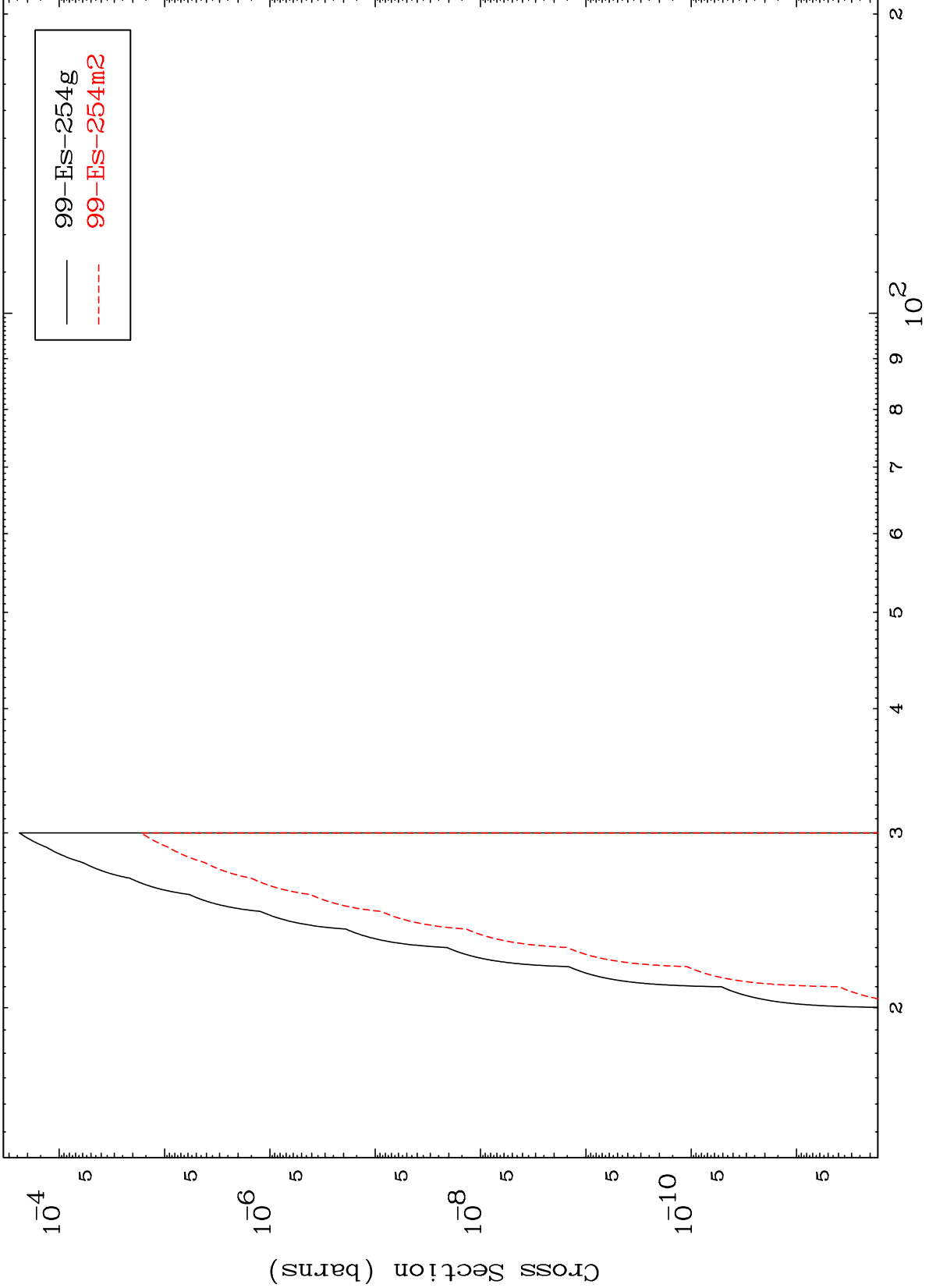


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(n,2n) d

100-Fm-257

Radionuclide Production Cross Section



29

Incident Energy (MeV)

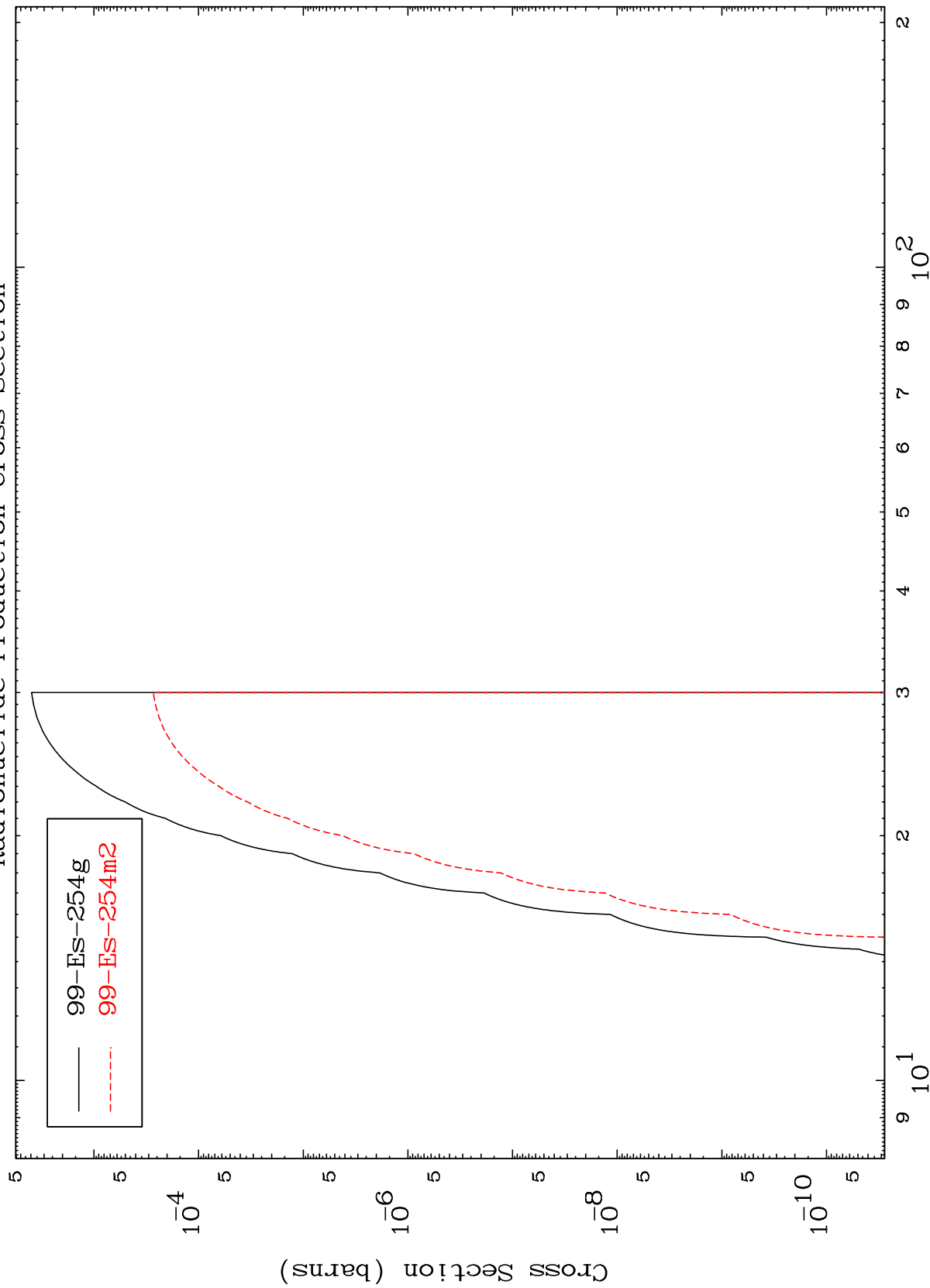
100-Fm-257

MAT 9938

(n,n') t

100-Fm-257

Radionuclide Production Cross Section



30

Incident Energy (MeV)

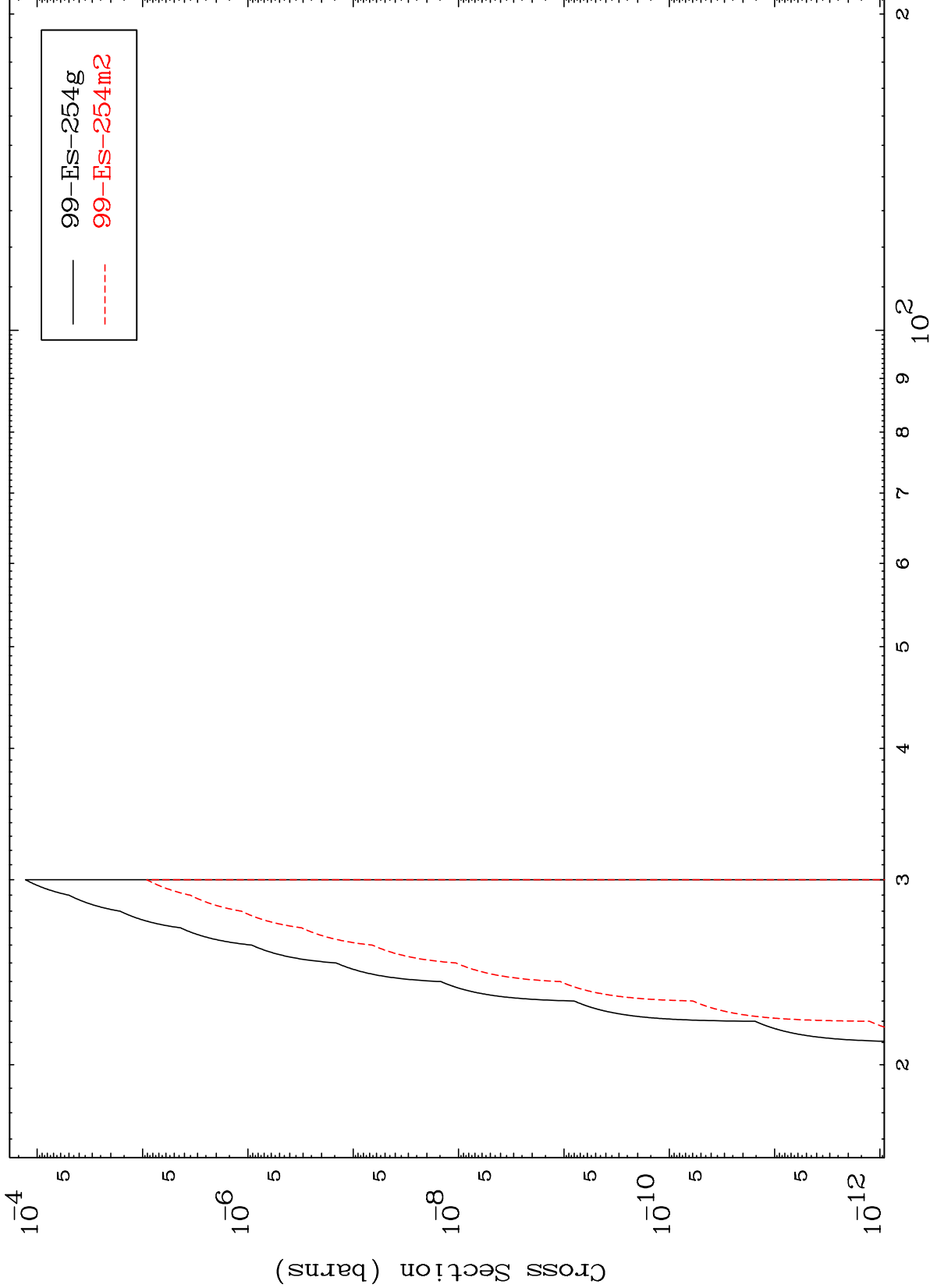
100-Fm-257

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(n,3n) p

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Radionuclide Production Cross Section



99-Es-254g
99-Es-254m2

Incident Energy (MeV)

100-Fm-257

31