

Program EVALPLOT
(Version 2021-1)

by

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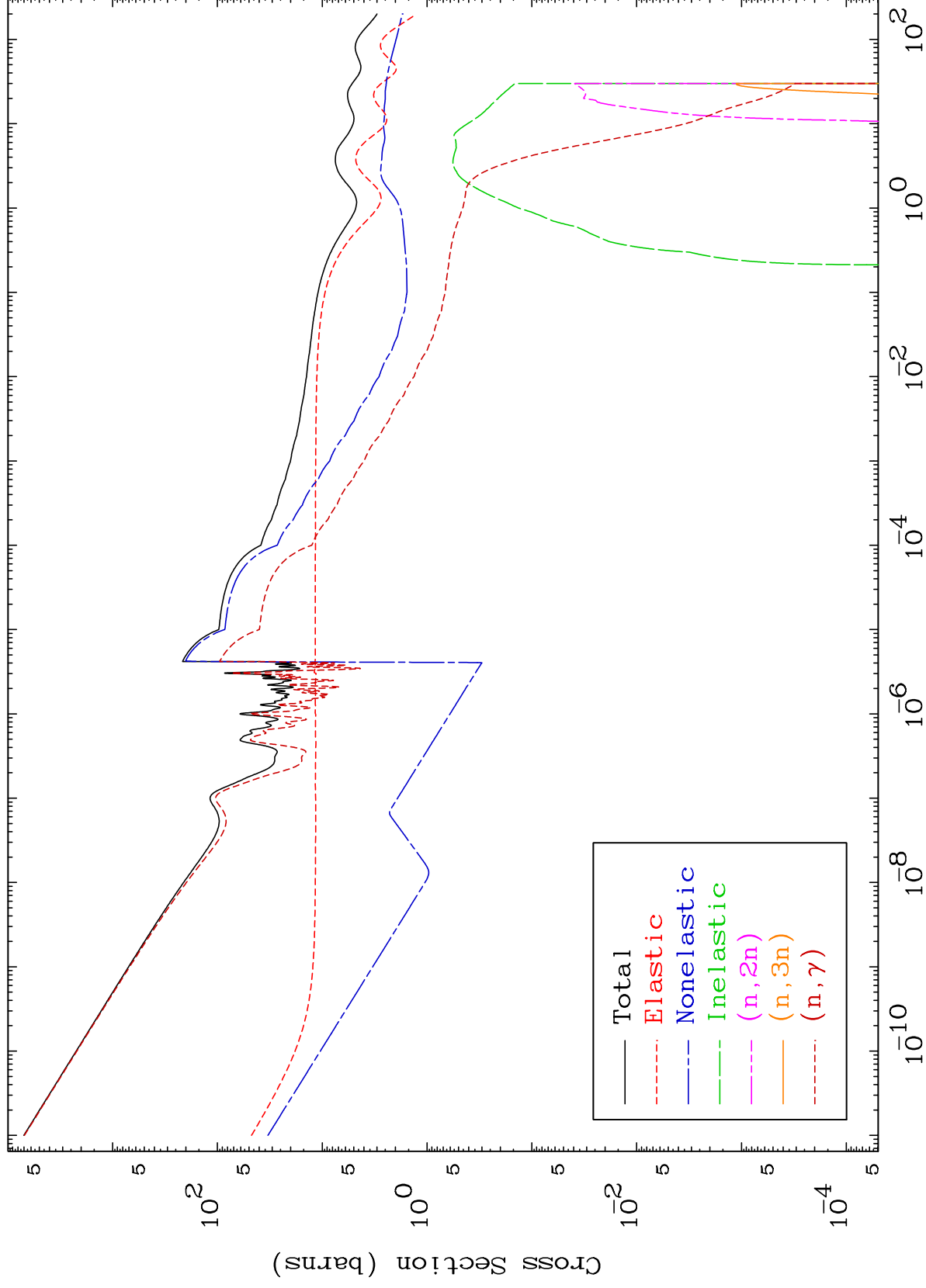
E.Mail:redcullen1@comcast.net
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 8704

Neutron Major
293 Kelvin Cross Sections

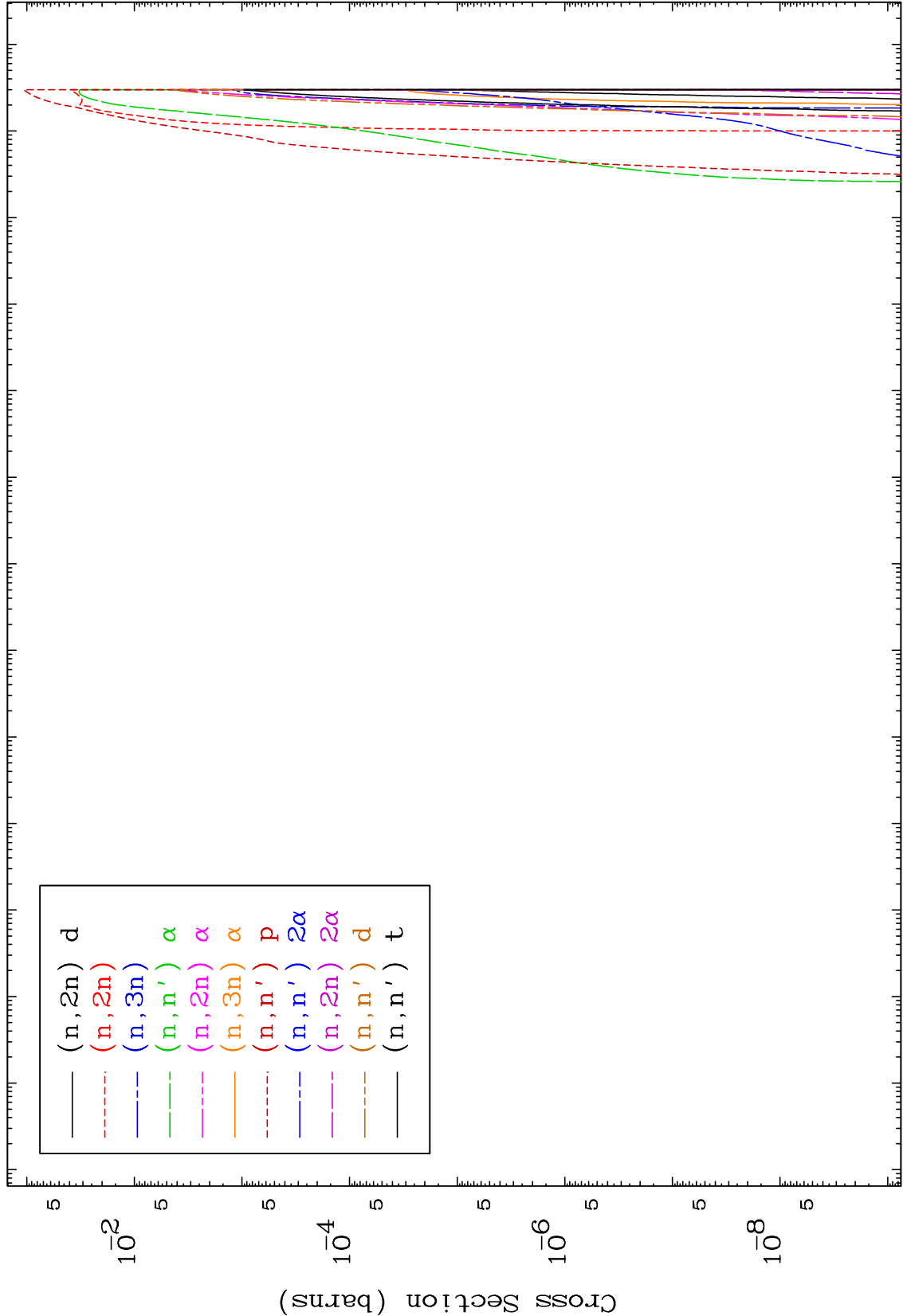
87-Fr-205

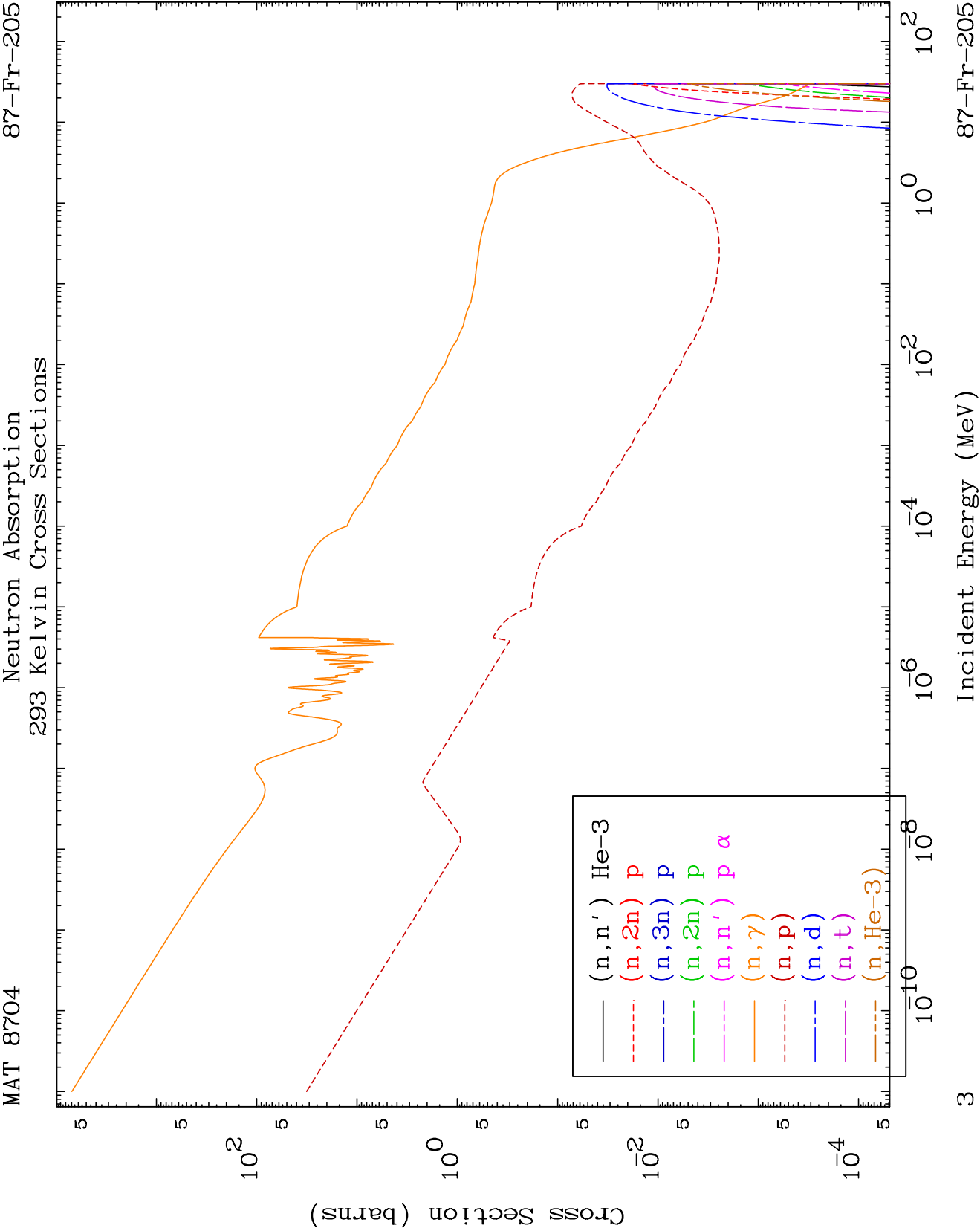


MAT 8704

Neutron Absorption
293 Kelvin Cross Sections

87-Fr-205

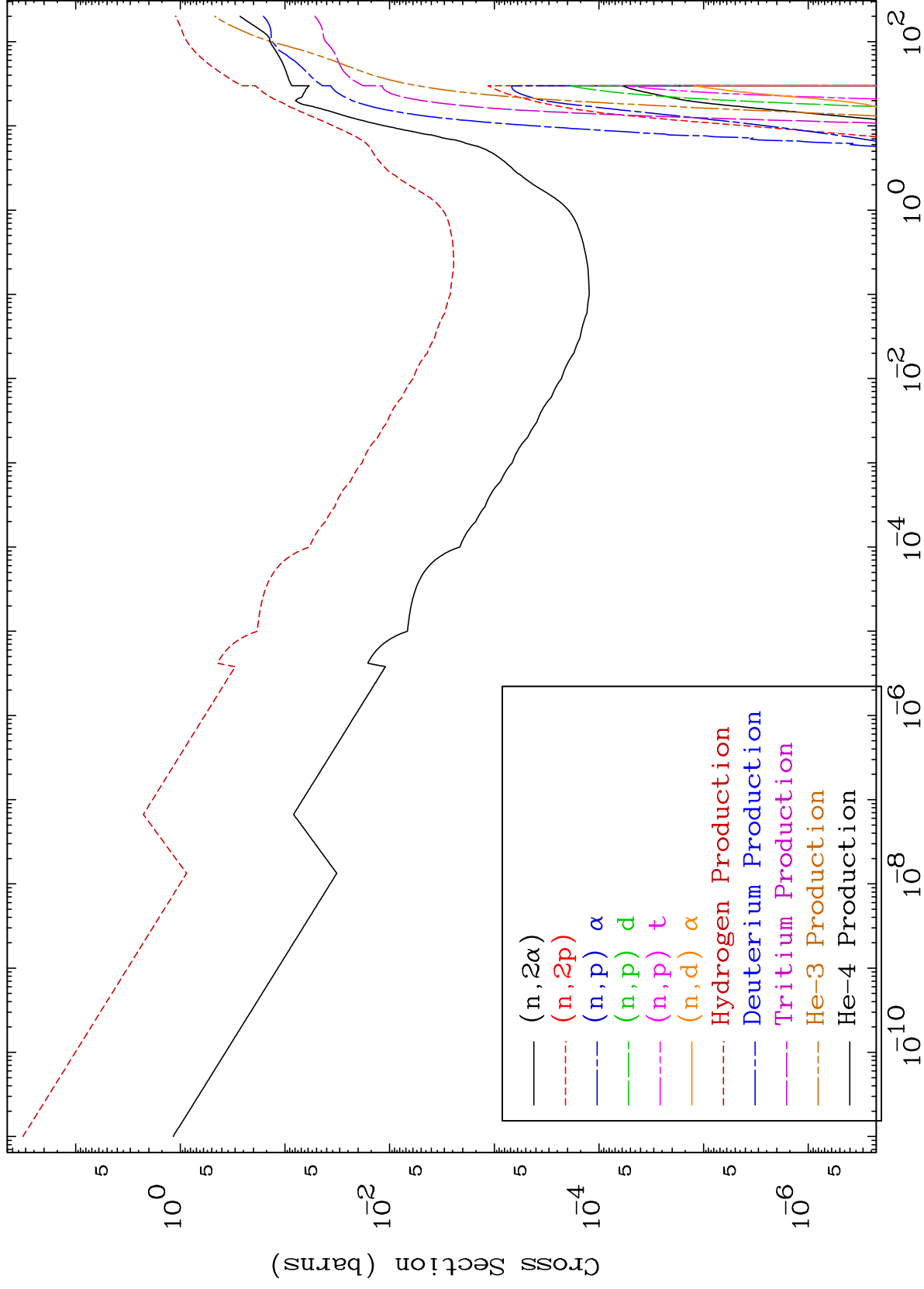




MAT 8704

Neutron Absorption
293 Kelvin Cross Sections

87-Fr-205



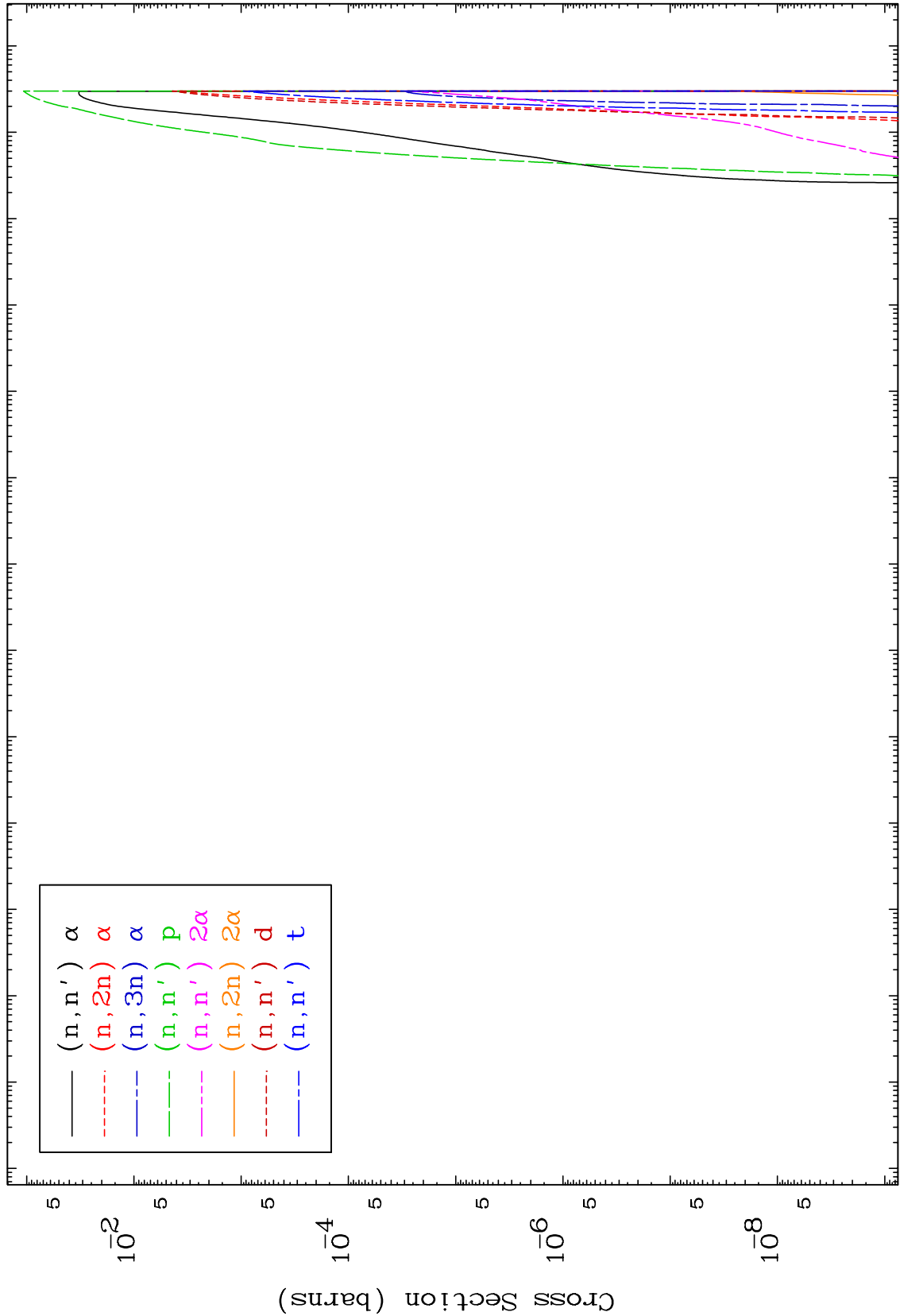
87-Fr-205

Incident Energy (MeV)

MAT 8704

Charged Particle
293 Kelvin Cross Sections

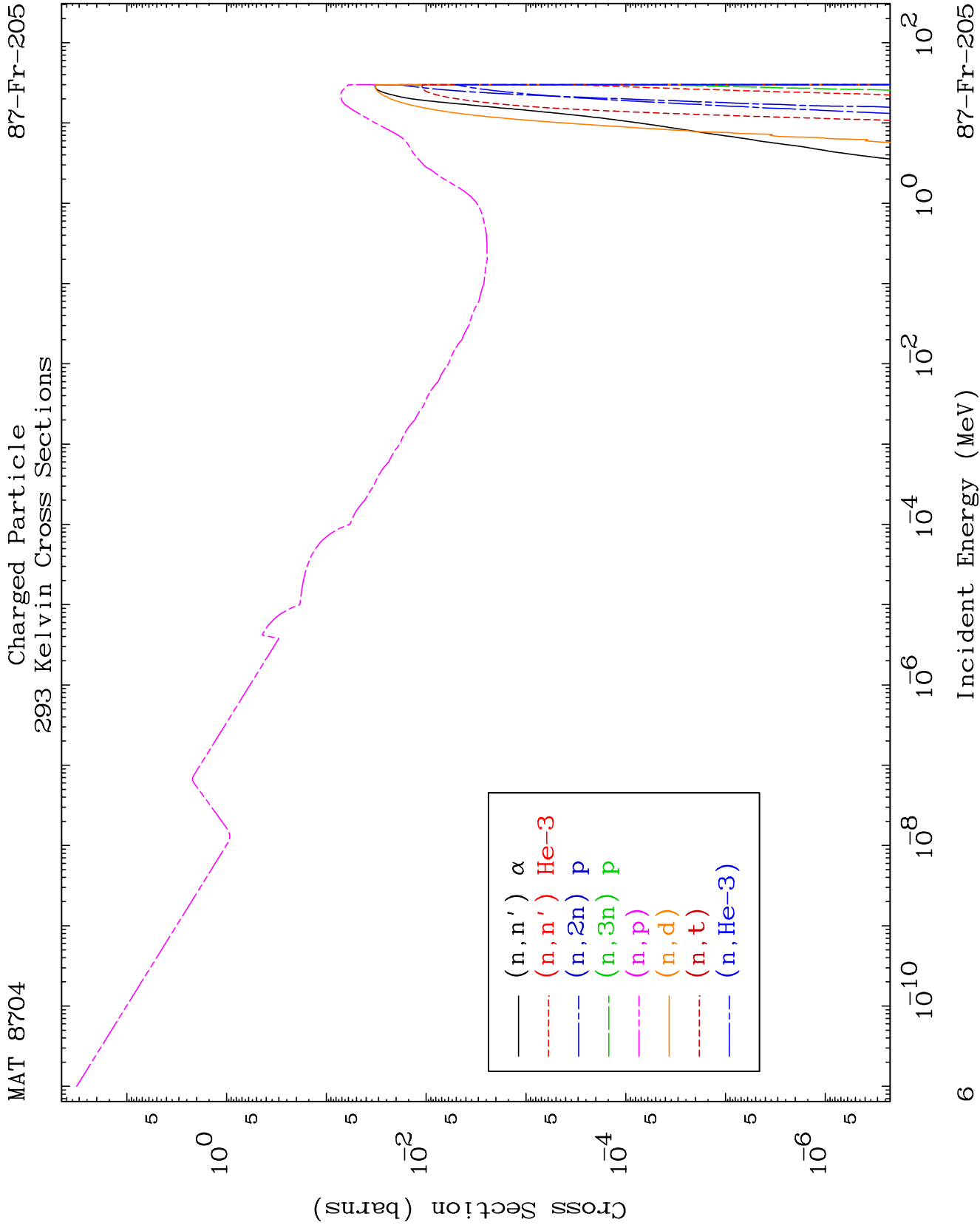
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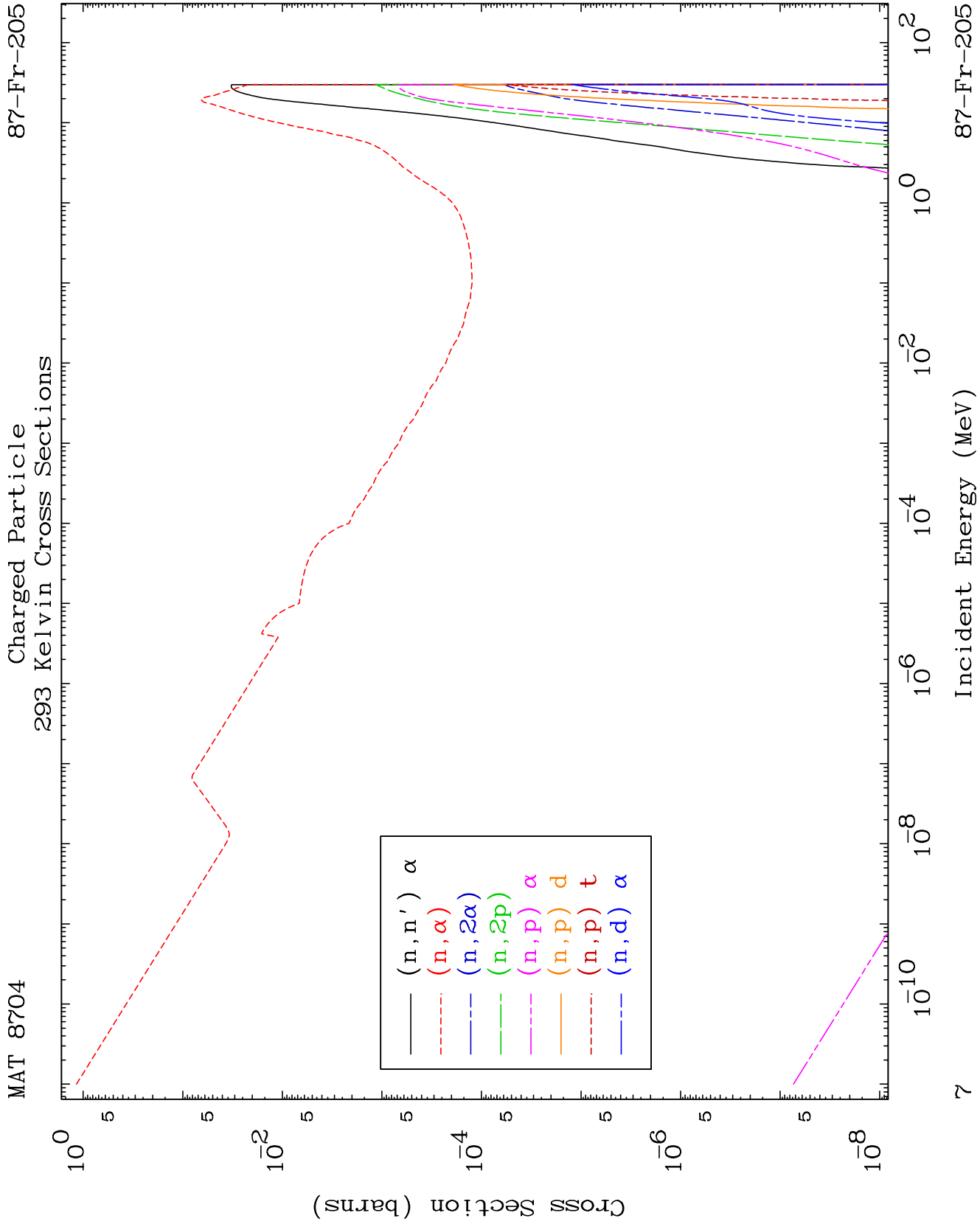


5

Incident Energy (MeV)

87-Fr-205

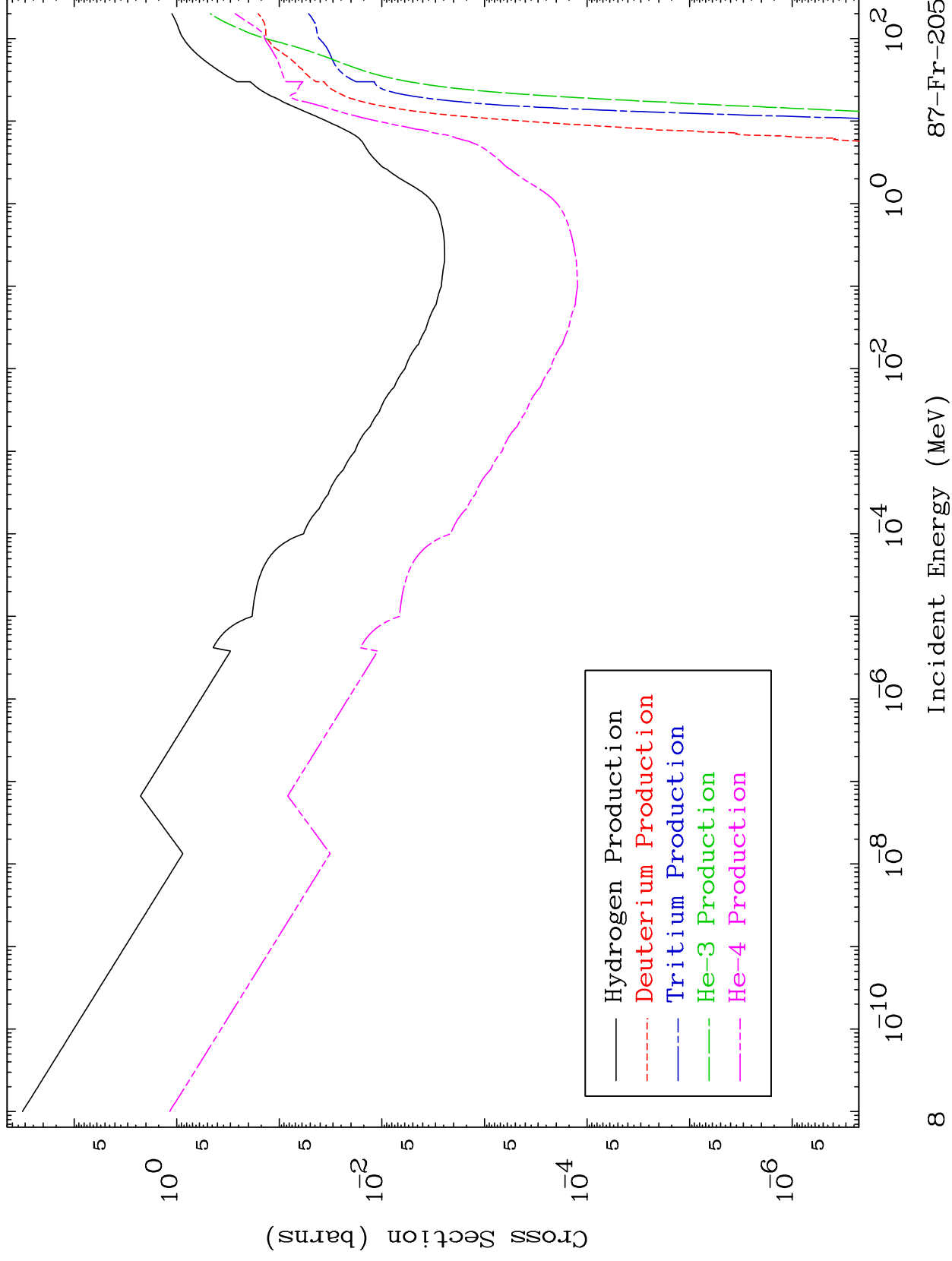


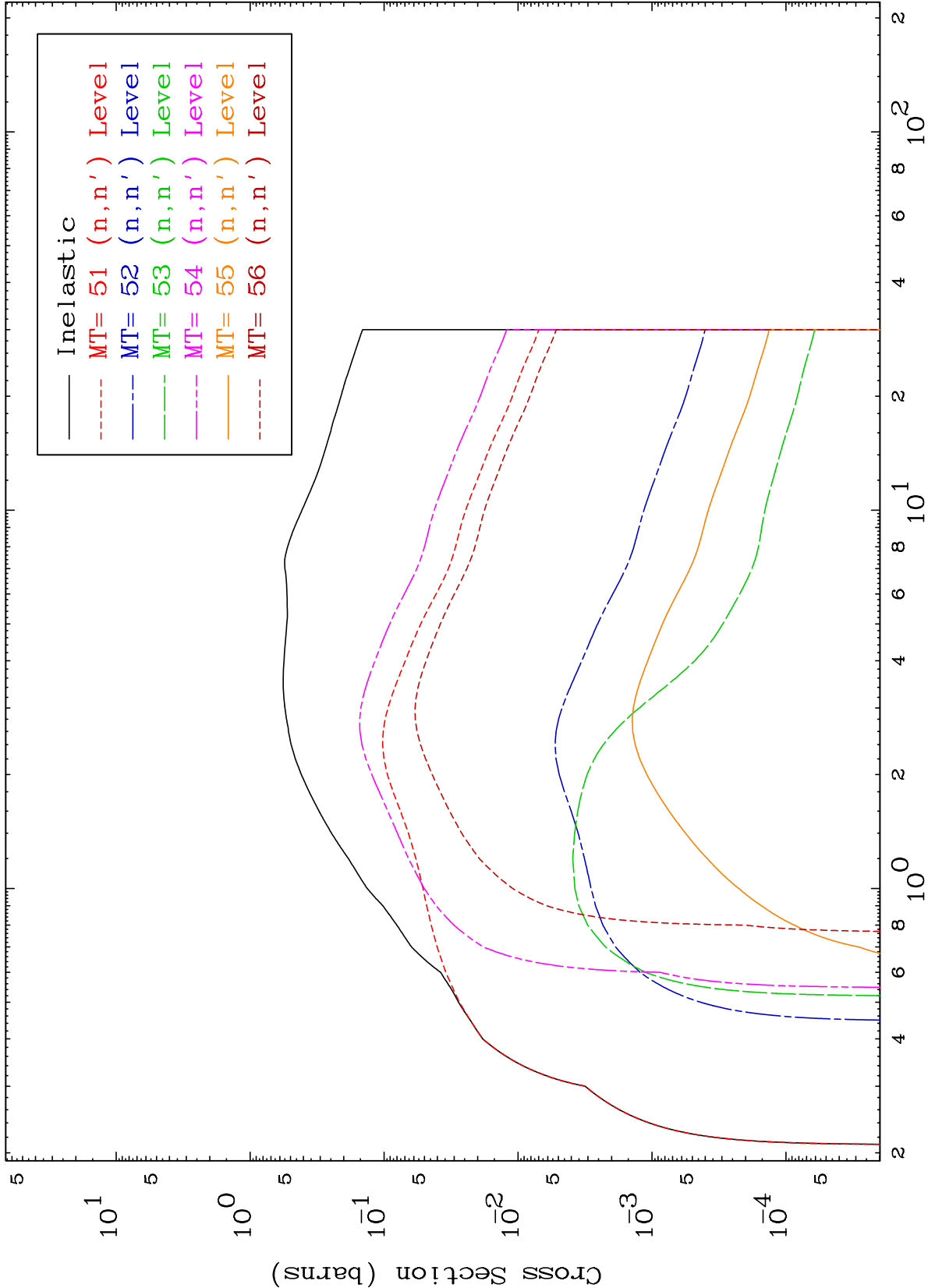


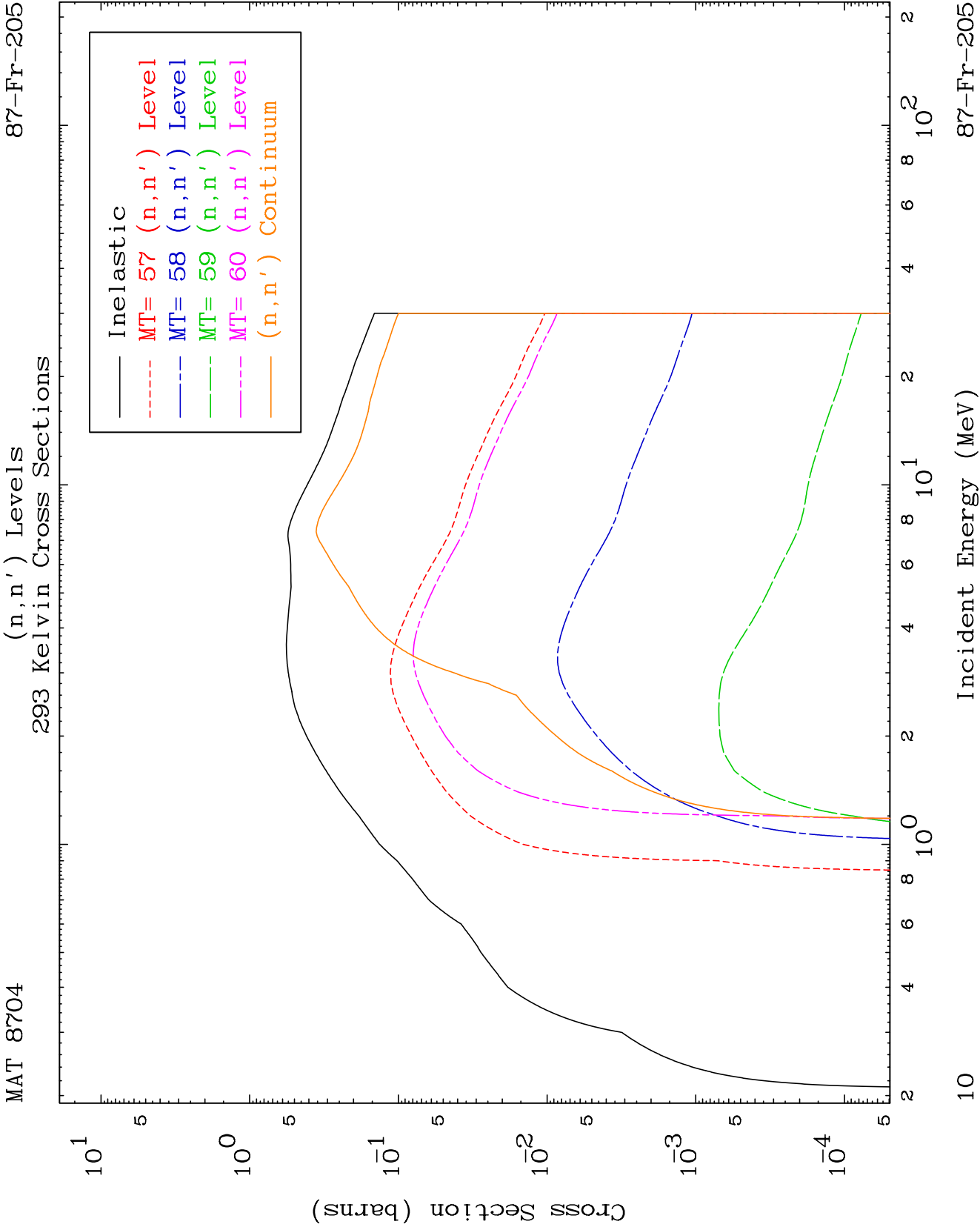
MAT 8704

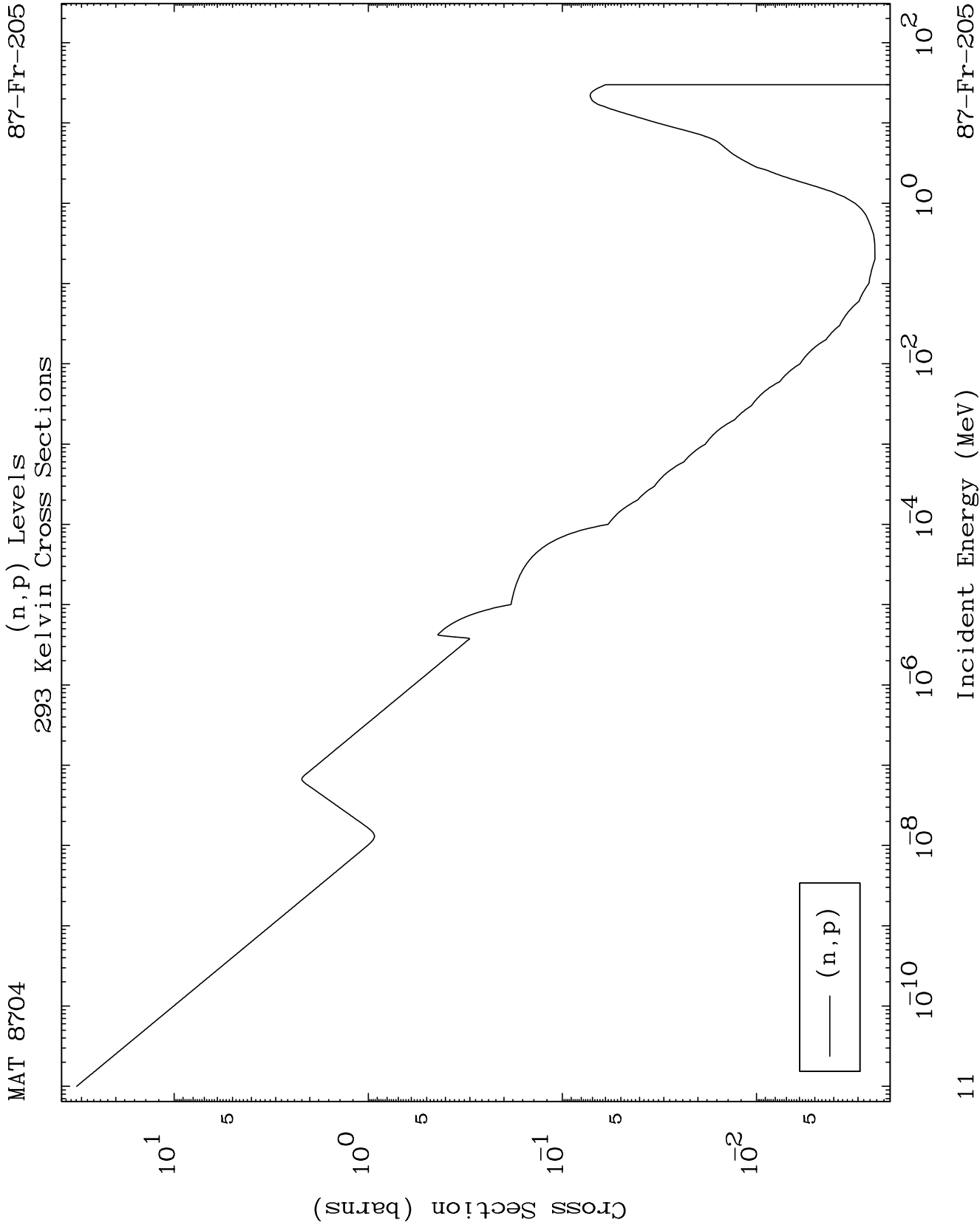
Particle Production
293 Kelvin Cross Sections

87-Fr-205





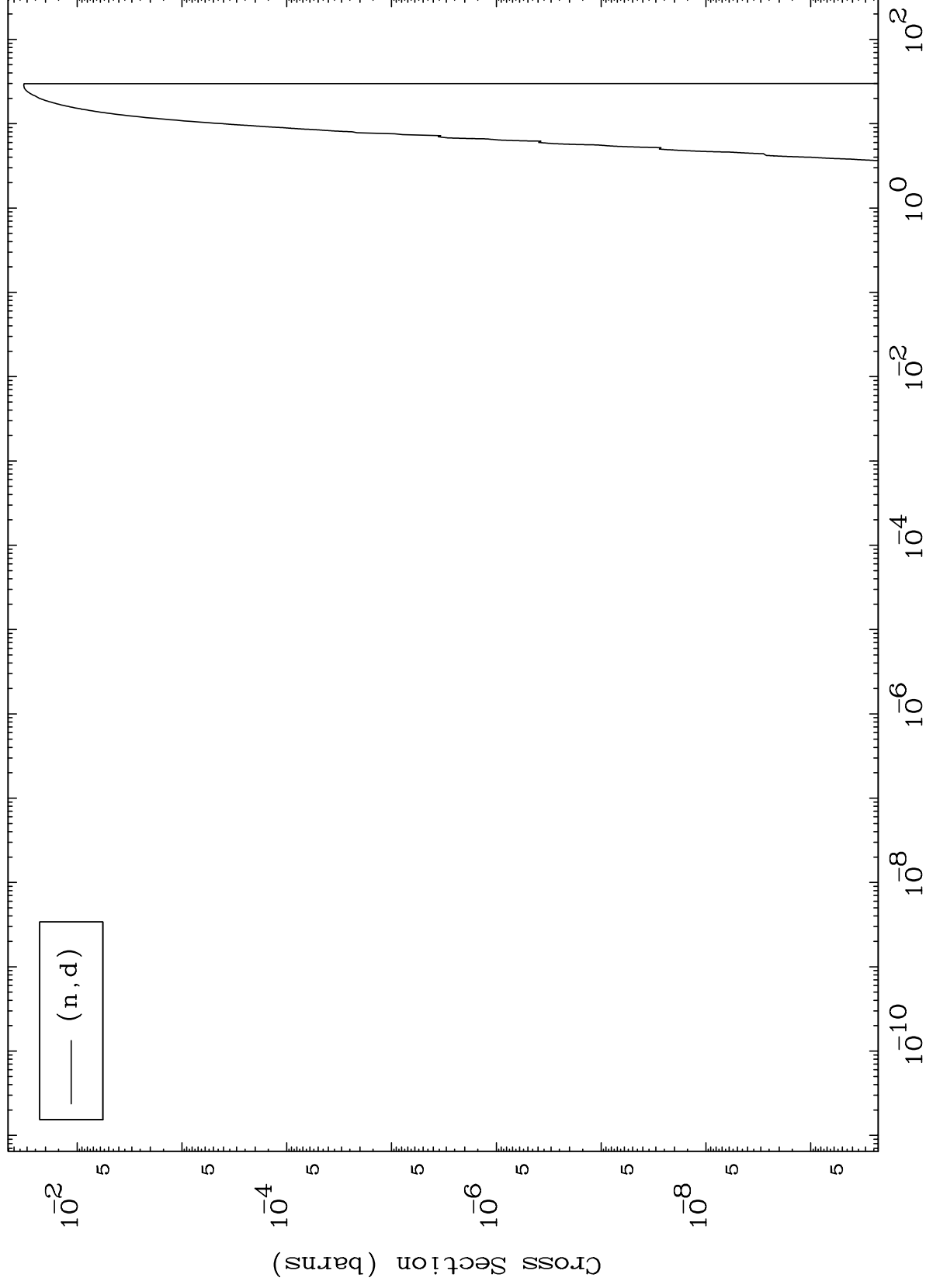




MAT 8704

(n,d) Levels
293 Kelvin Cross Sections

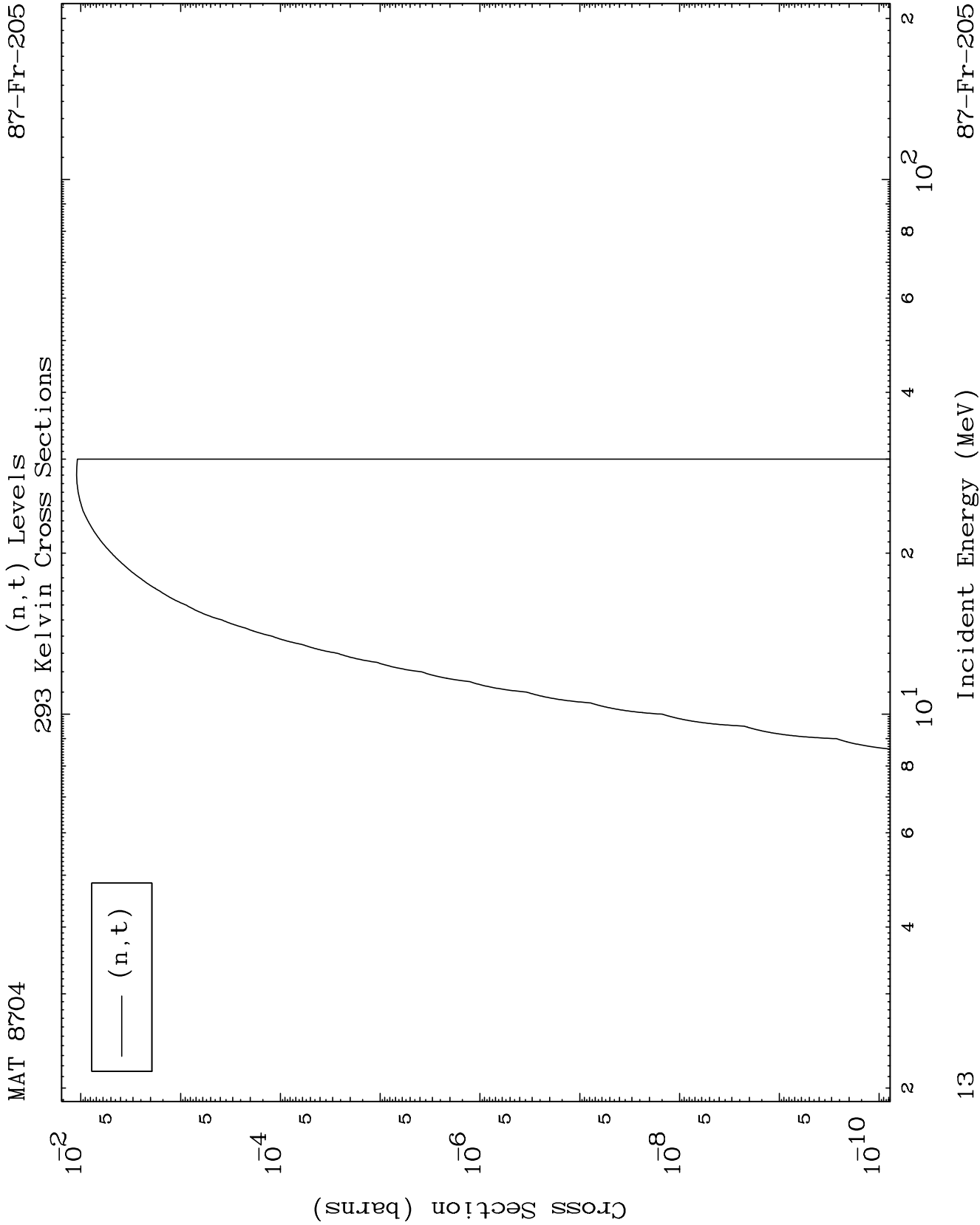
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Incident Energy (MeV)

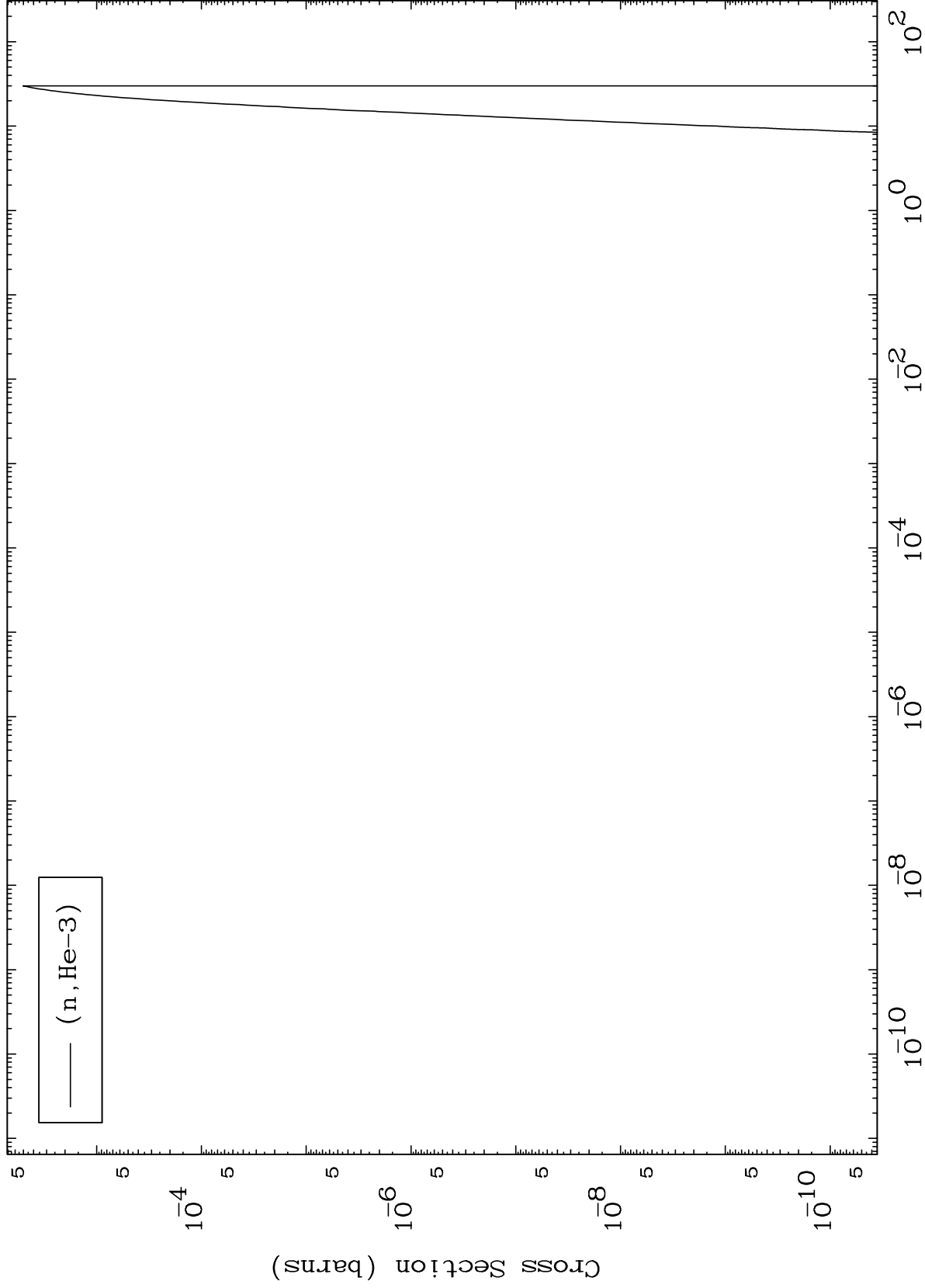
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MAT 8704

(n,He3) Levels
293 Kelvin Cross Sections

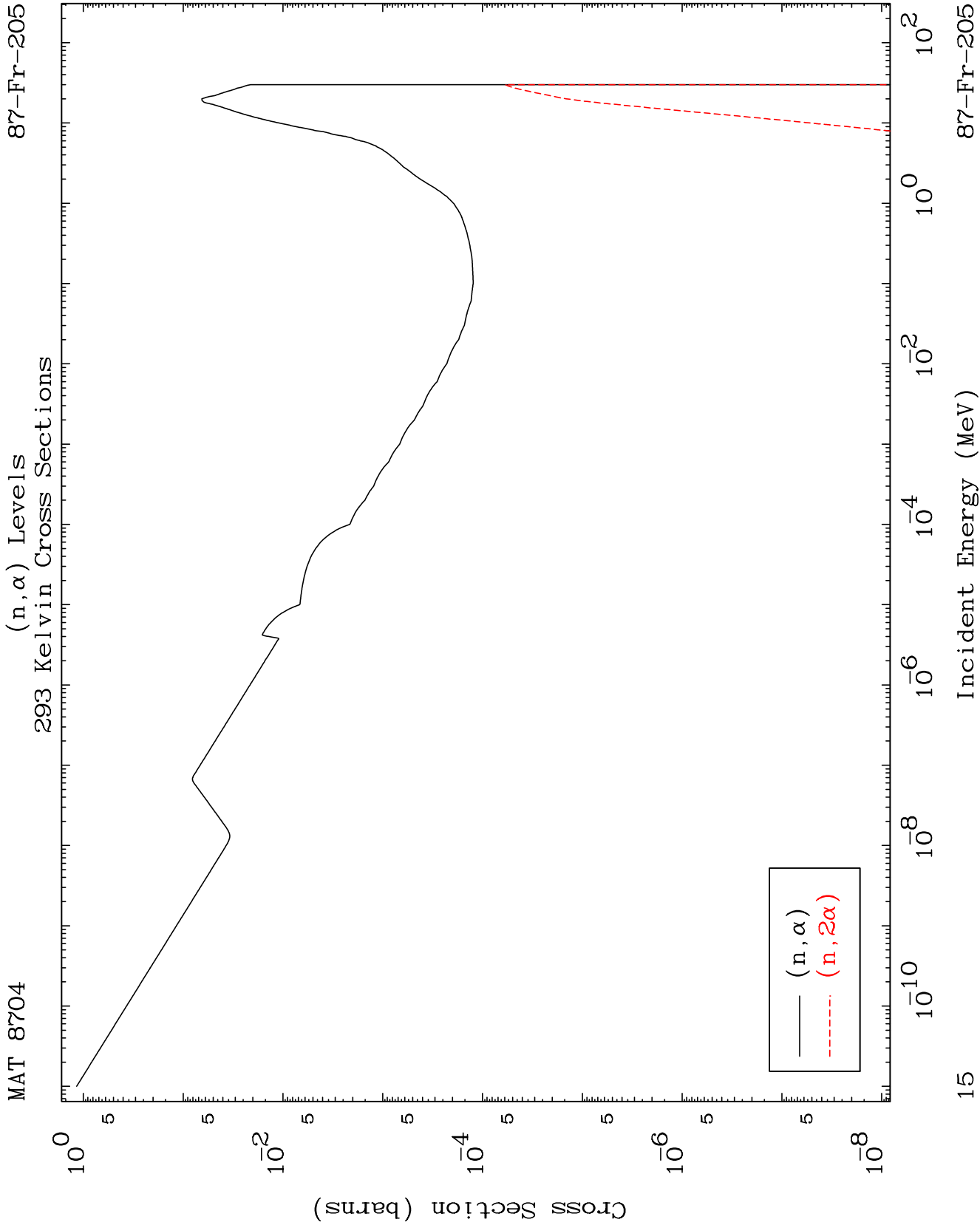
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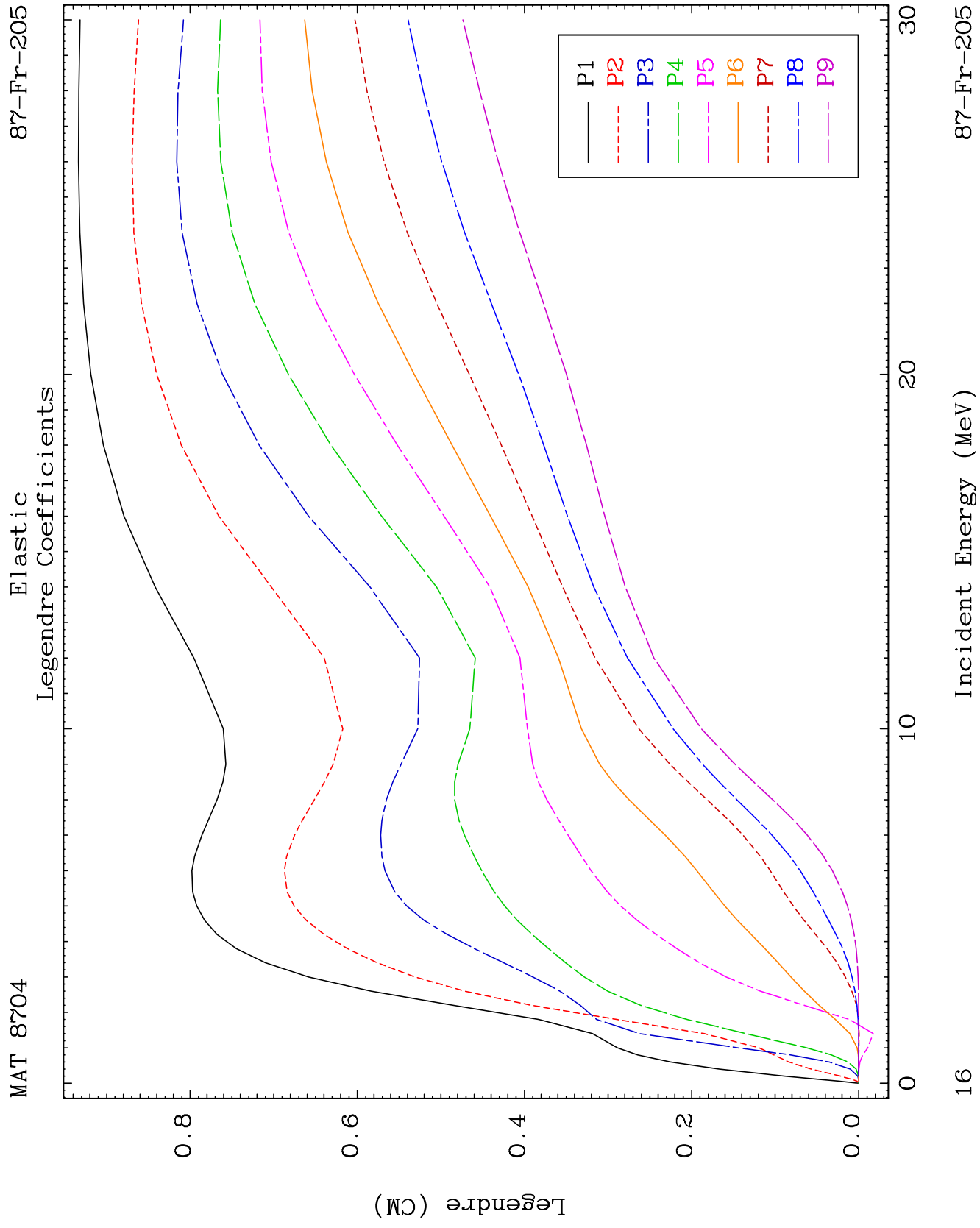


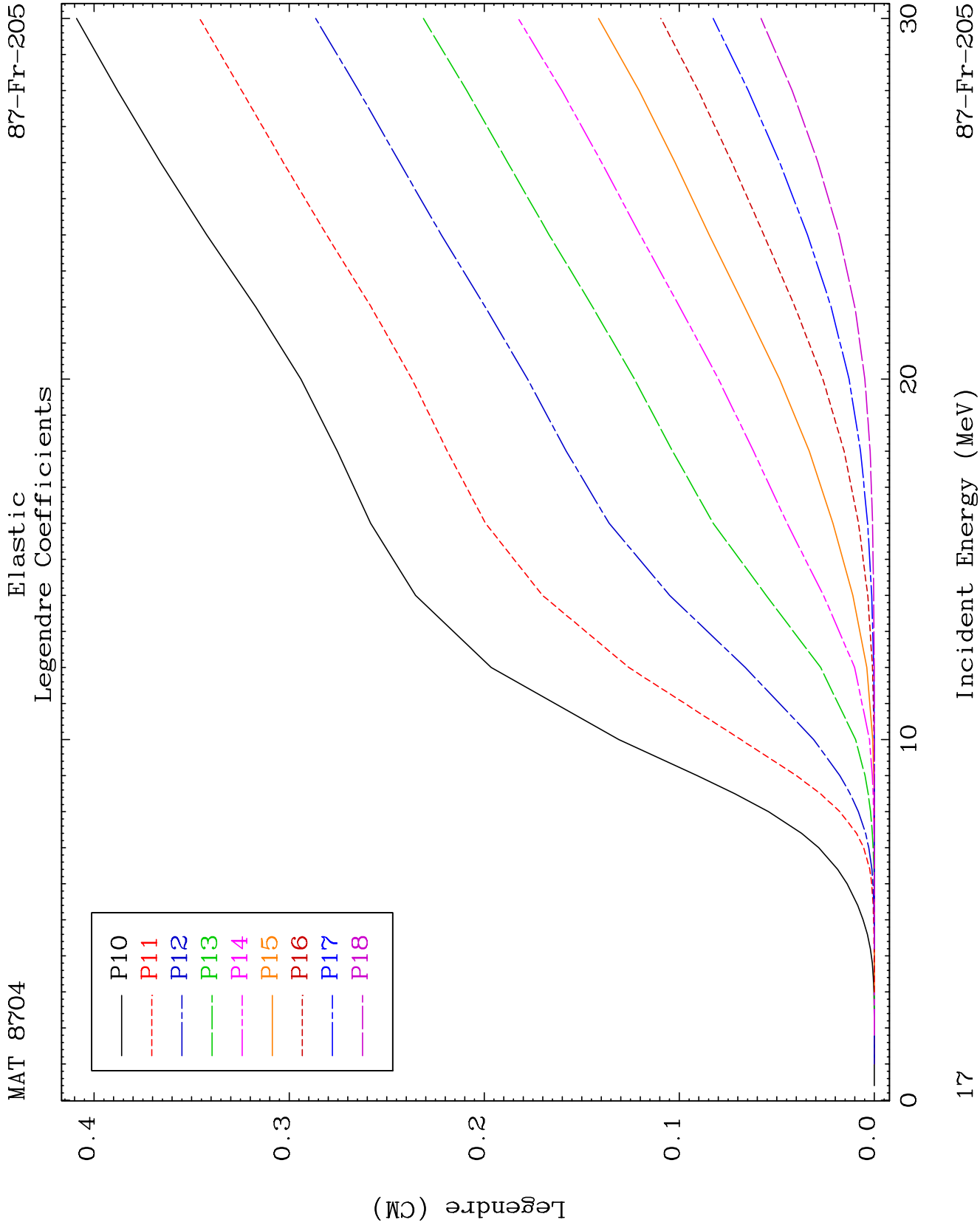
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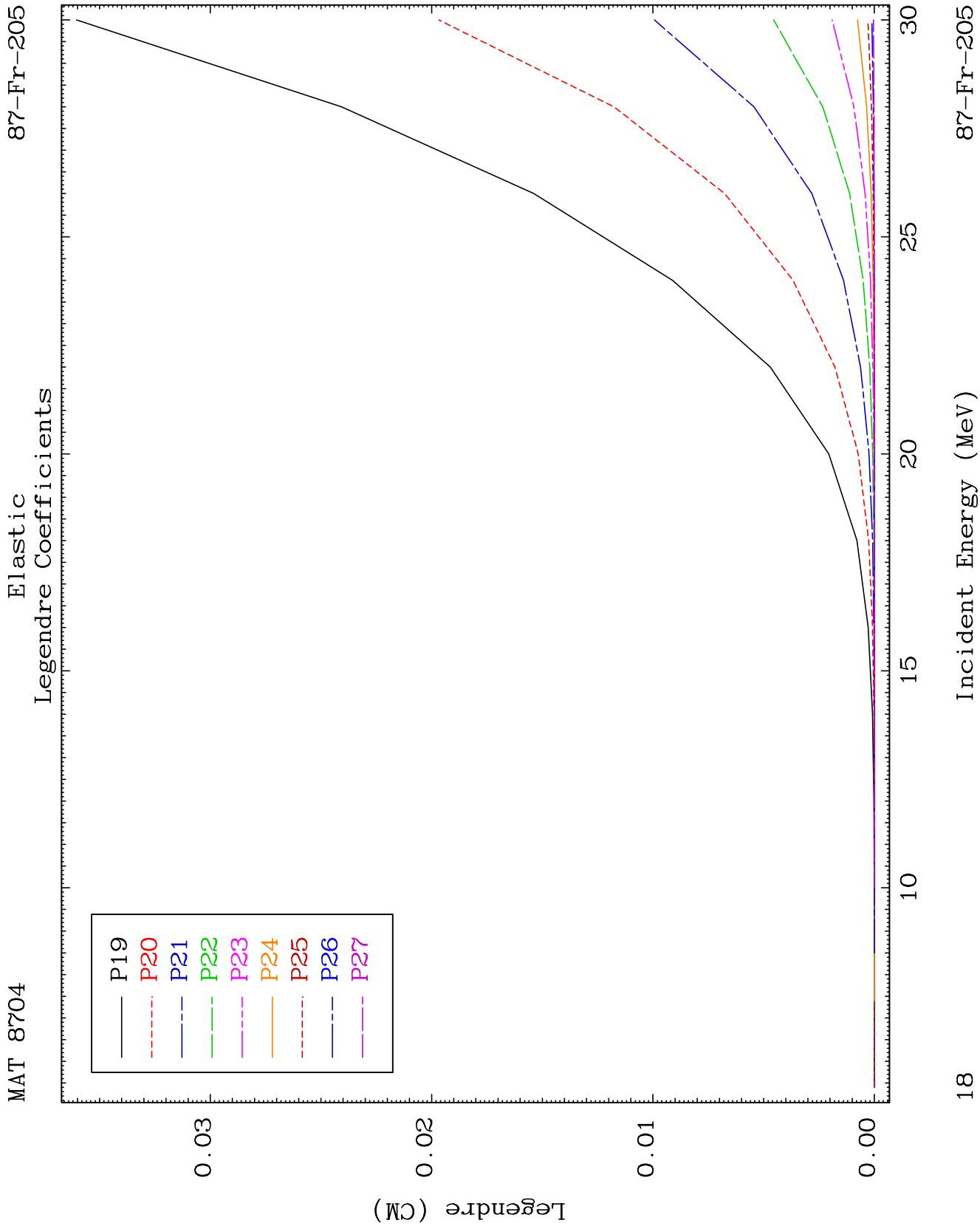
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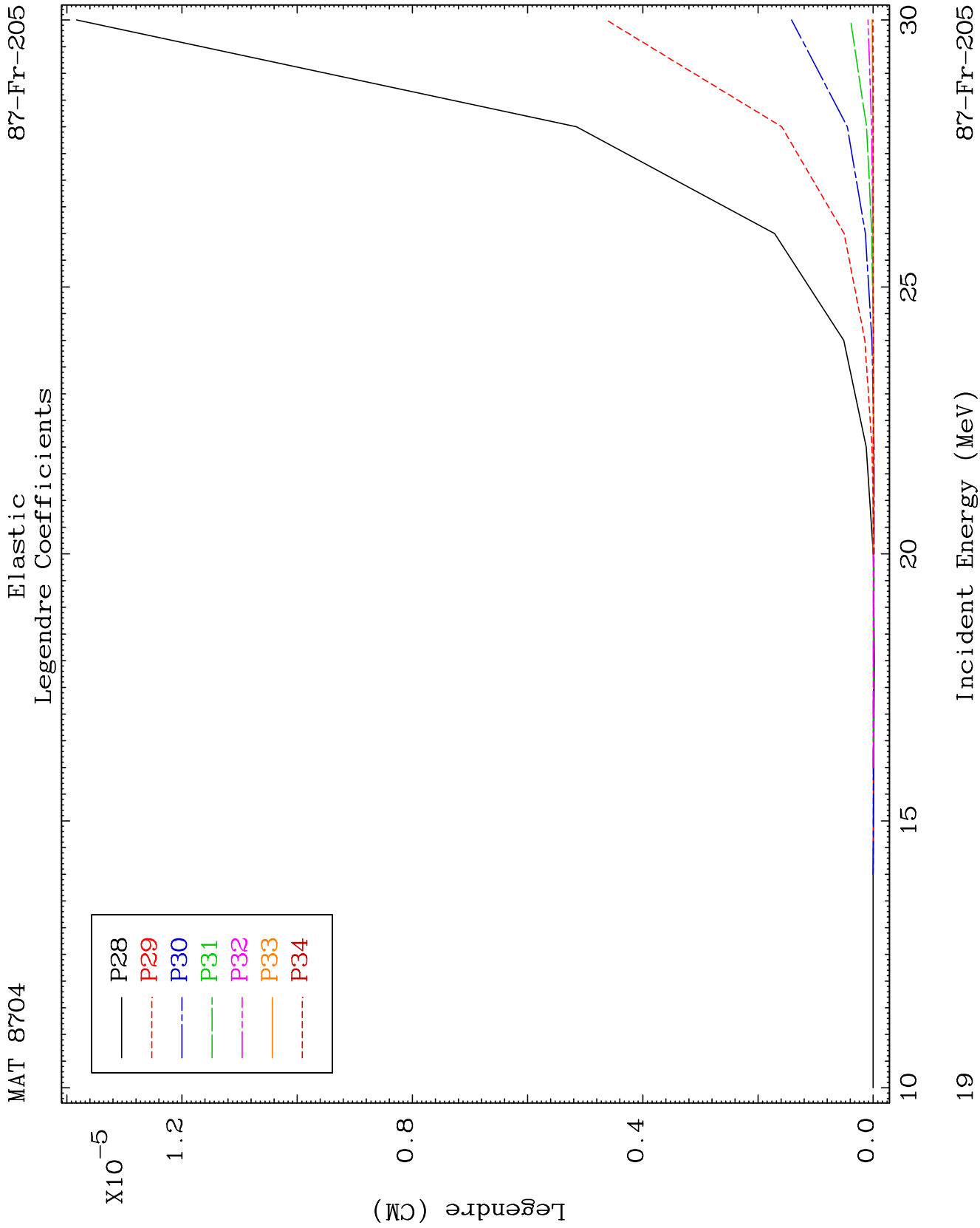
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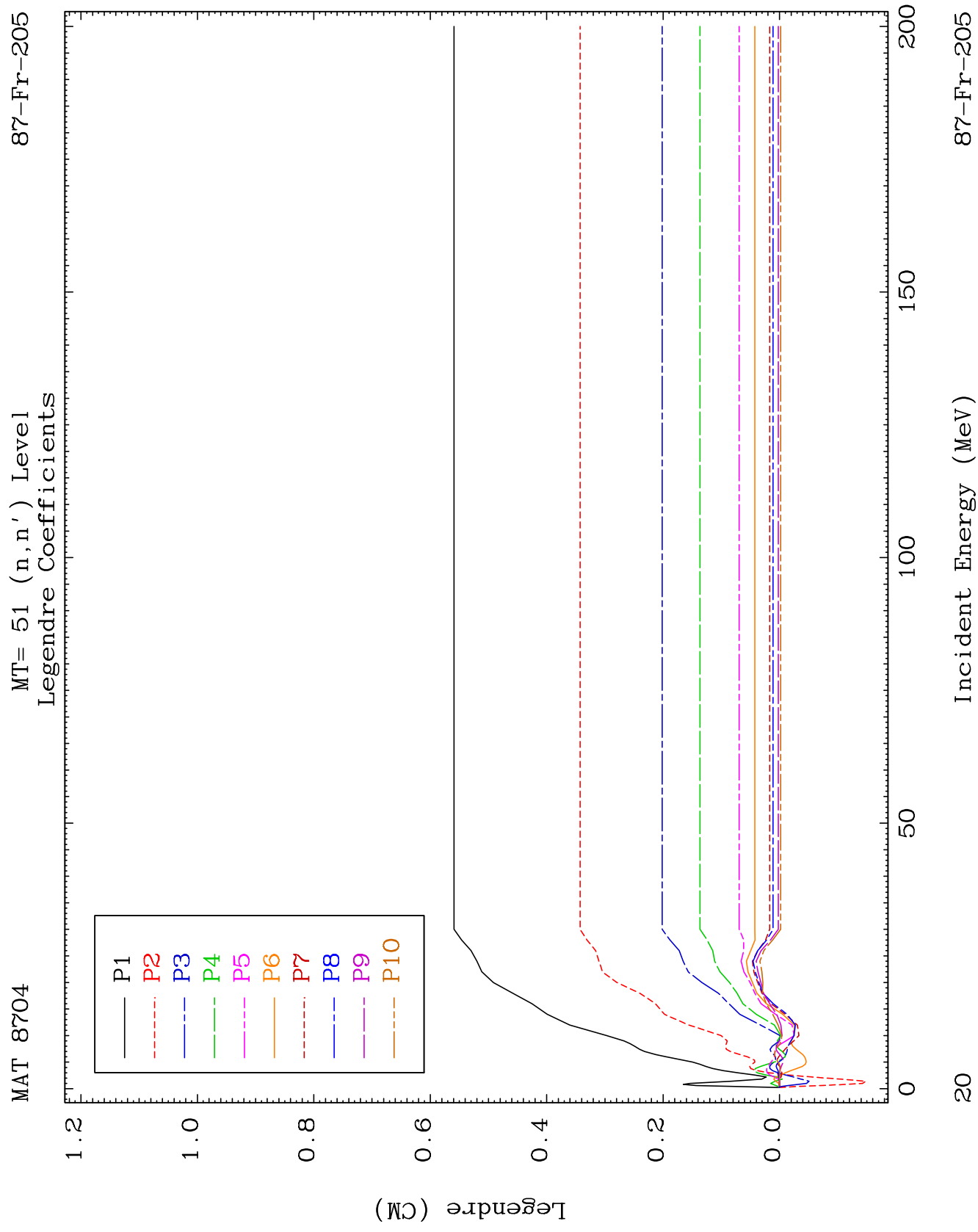








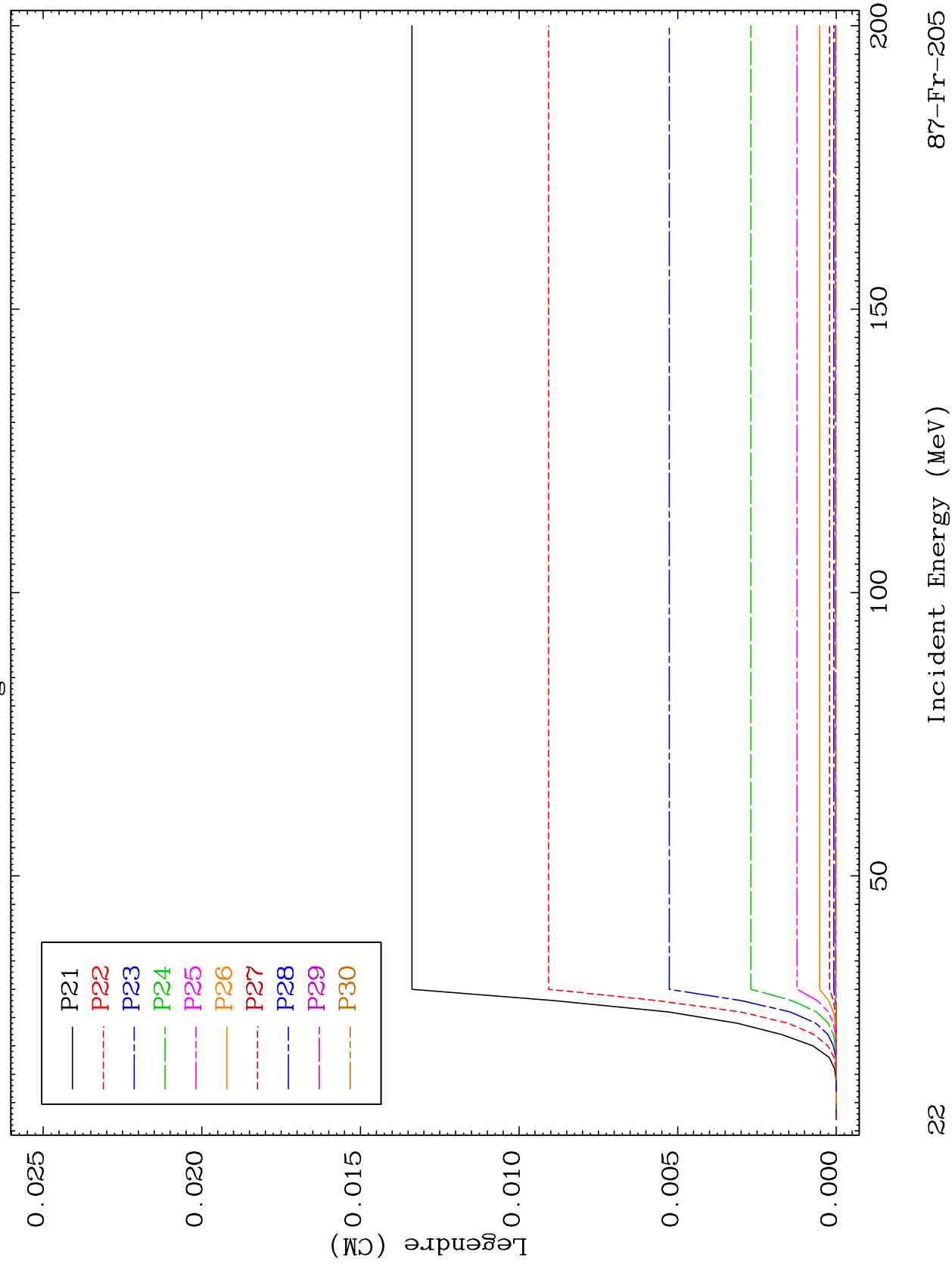




MAT 8704

MT= 51 (n, n') Level
Legendre Coefficients

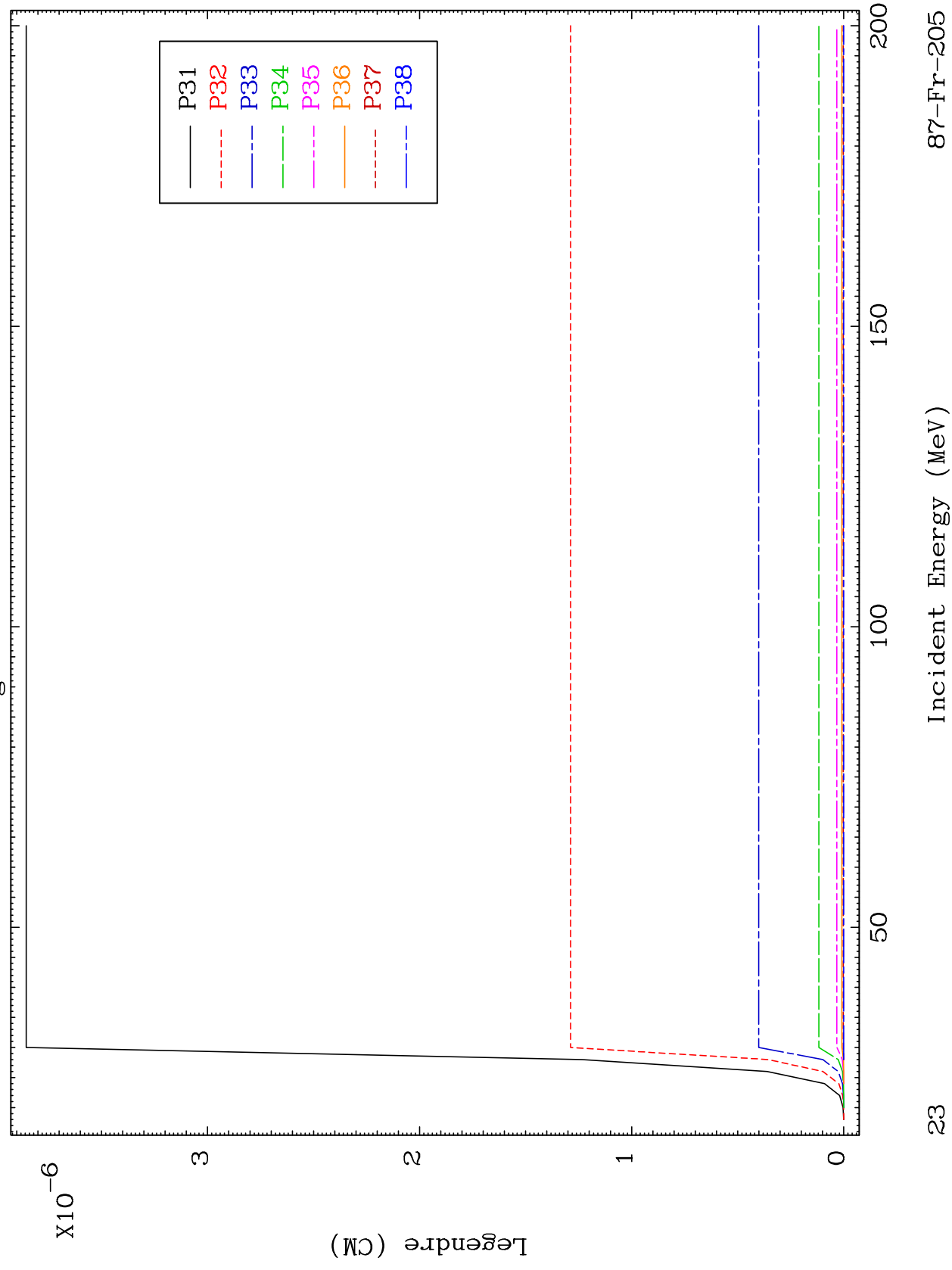
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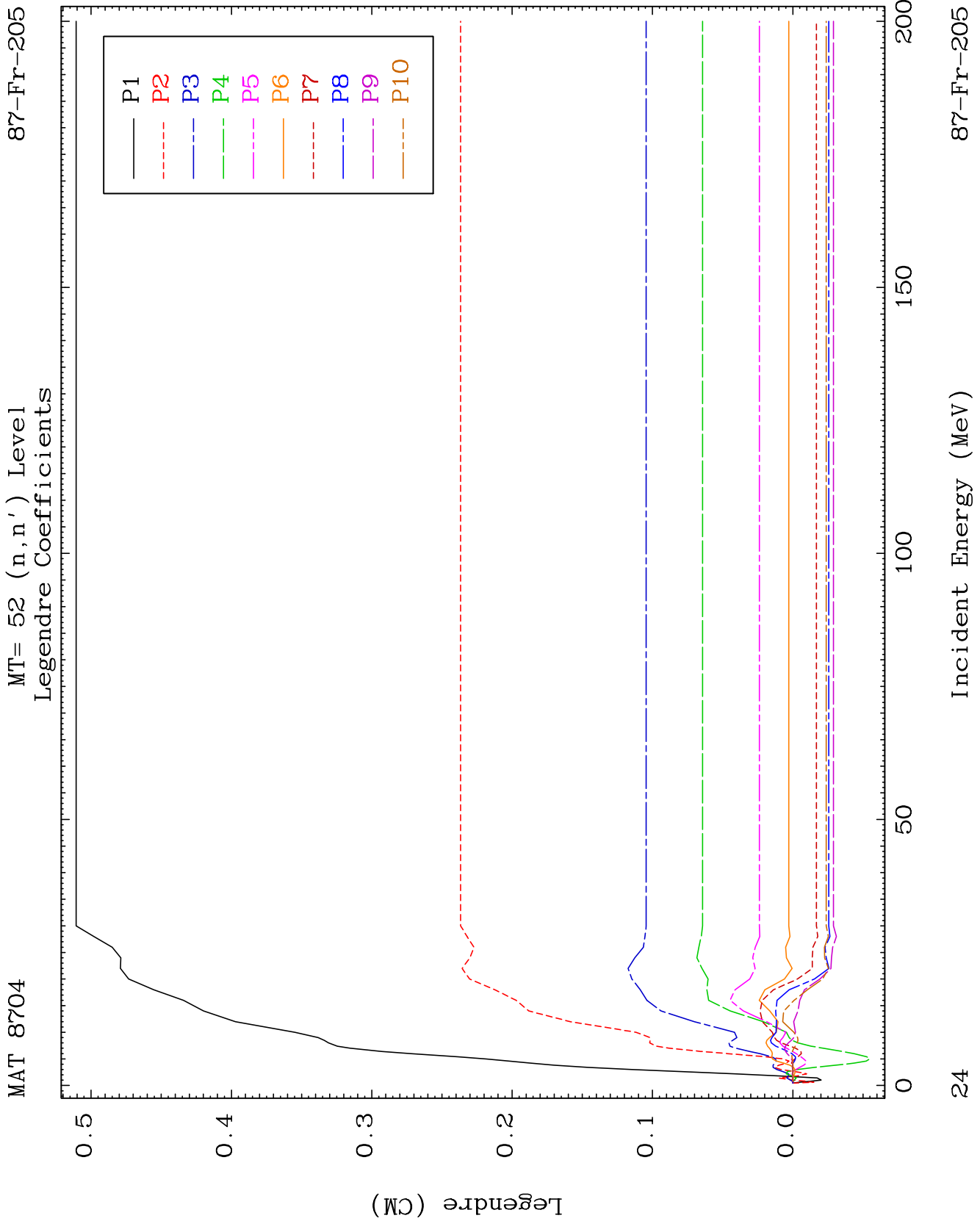


MAT 8704

MT= 51 (n, n') Level
Legendre Coefficients

87-Fr-205

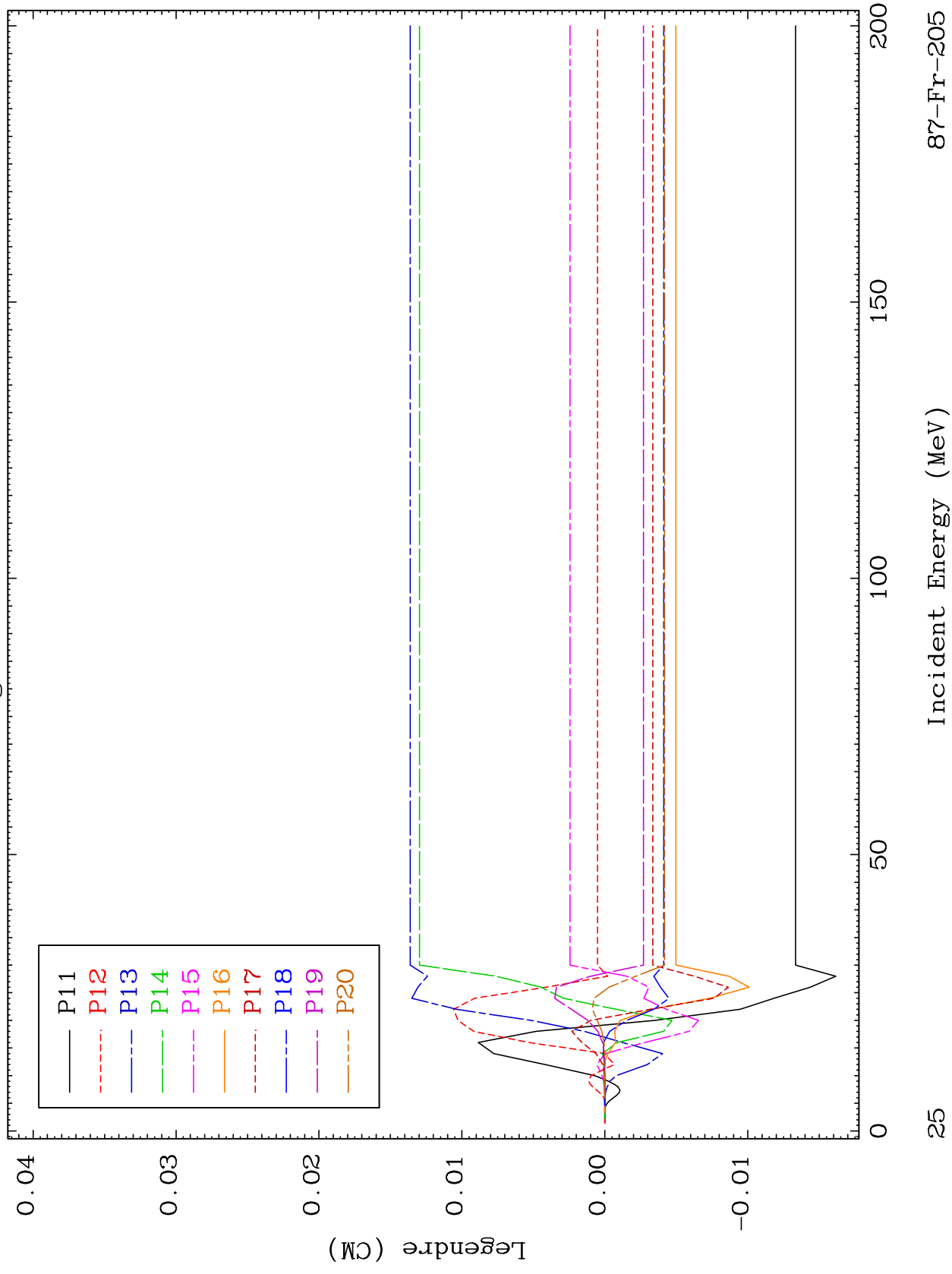




MAT 8704

MT= 52 (n, n') Level
Legendre Coefficients

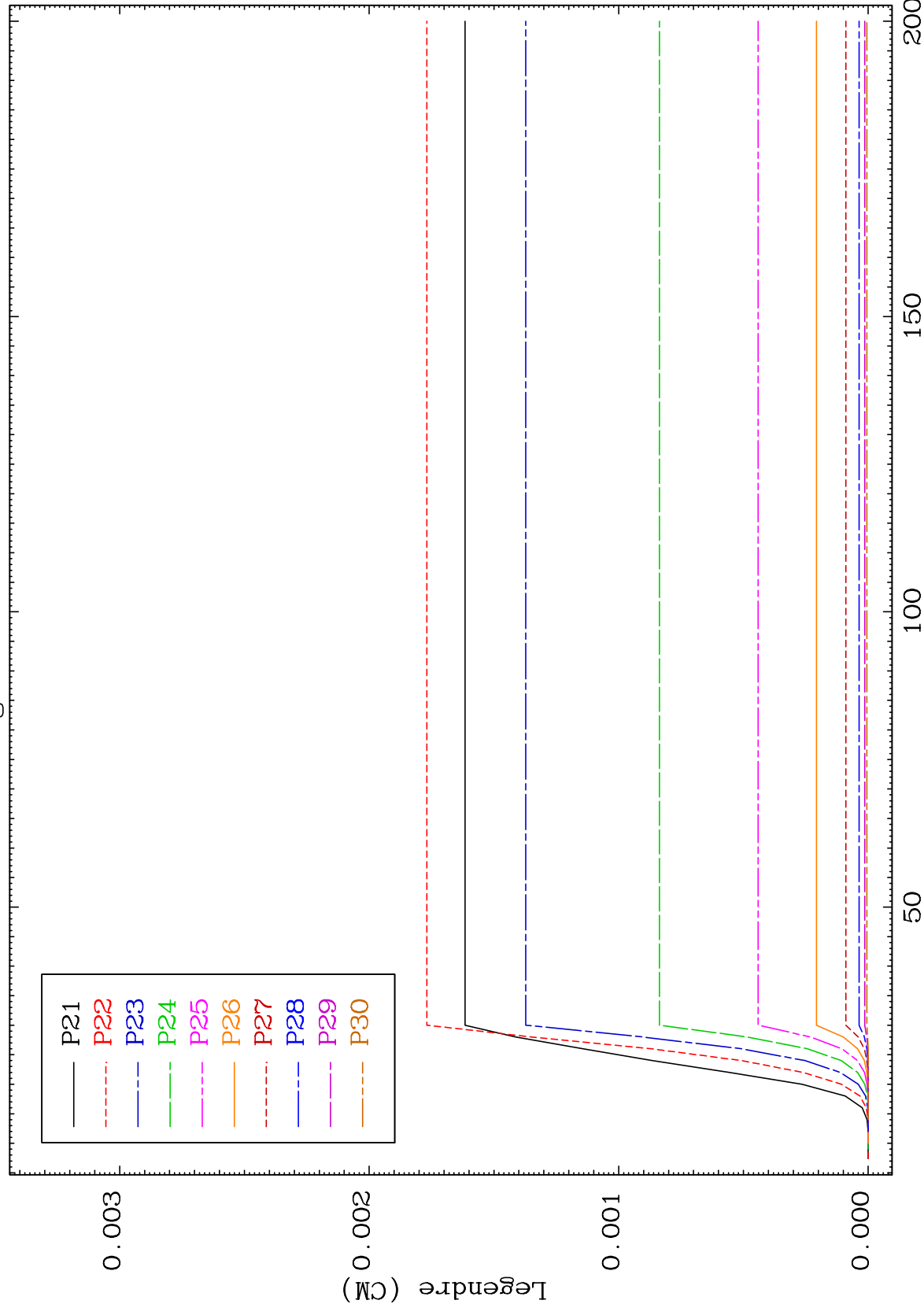
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MAT 8704

MT= 52 (n, n') Level
Legendre Coefficients

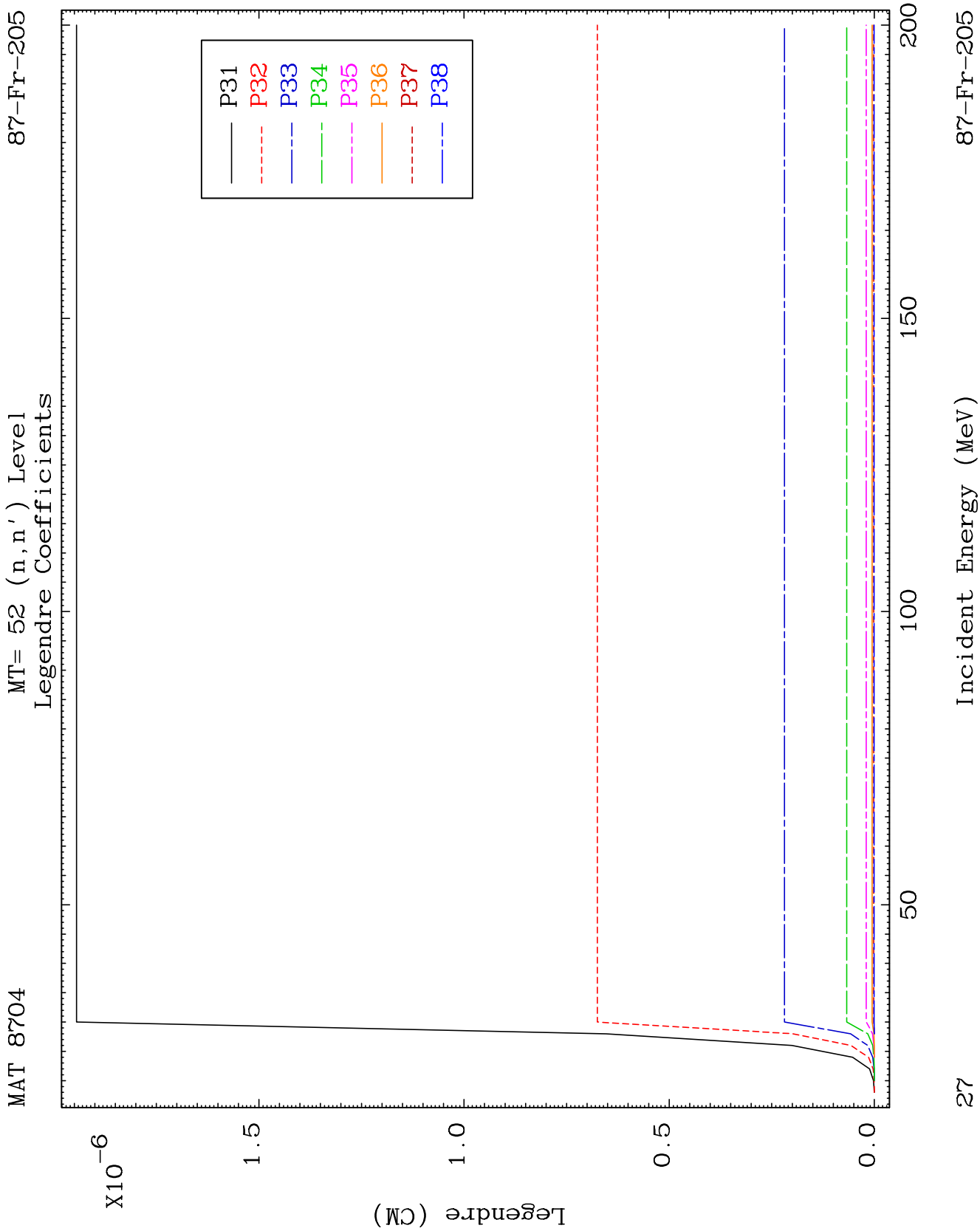
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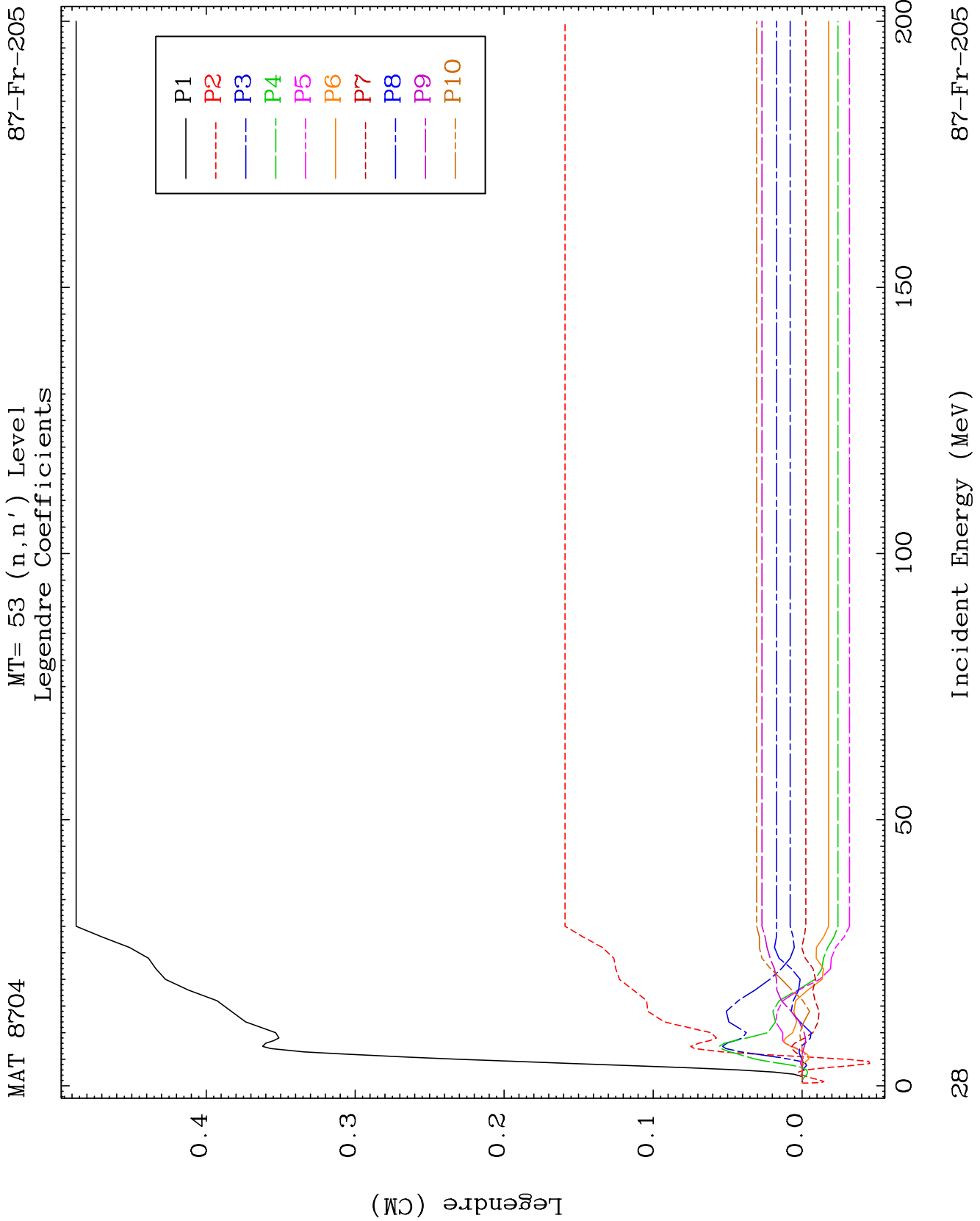


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Incident Energy (MeV)

87-FR-205

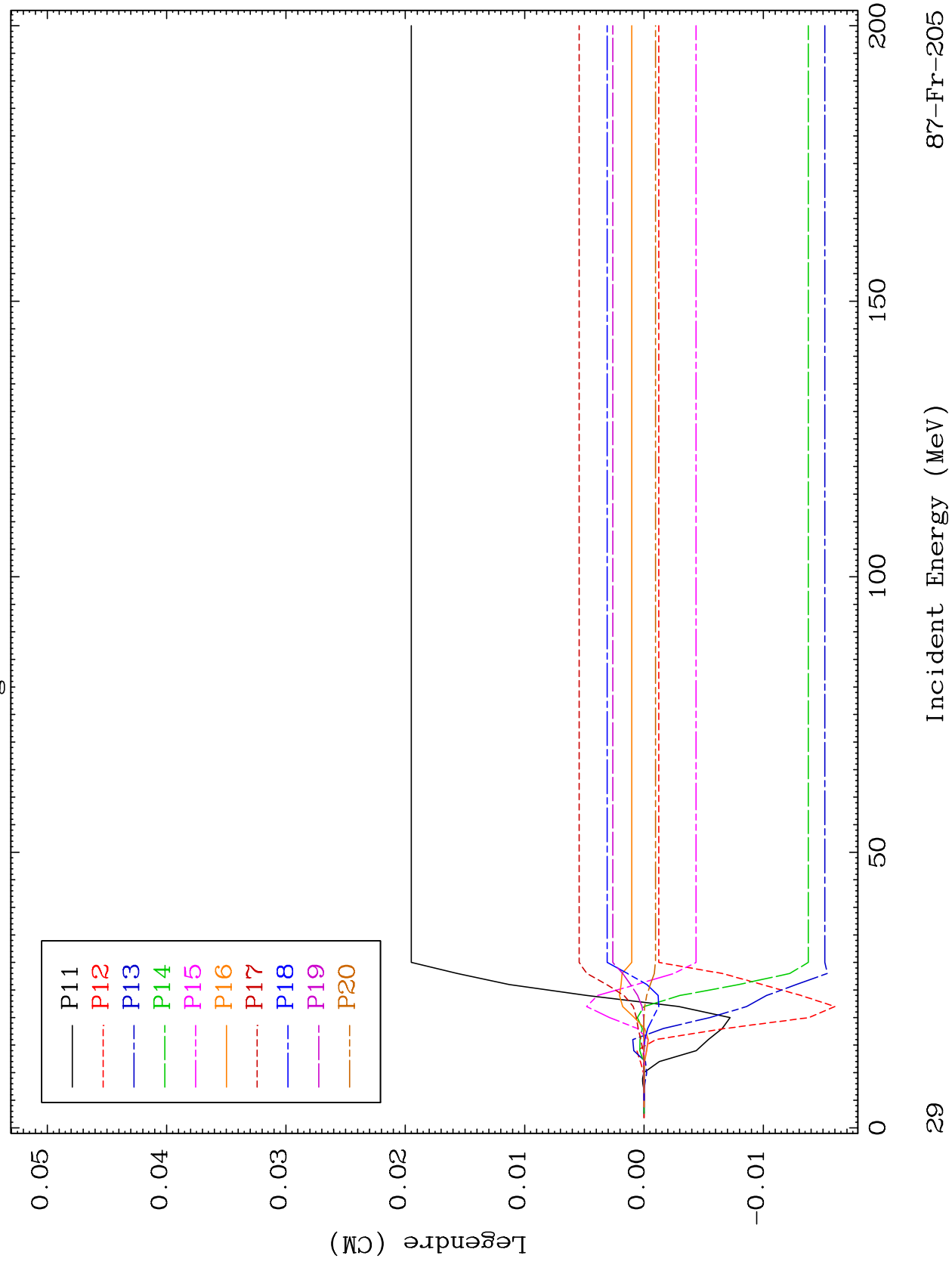


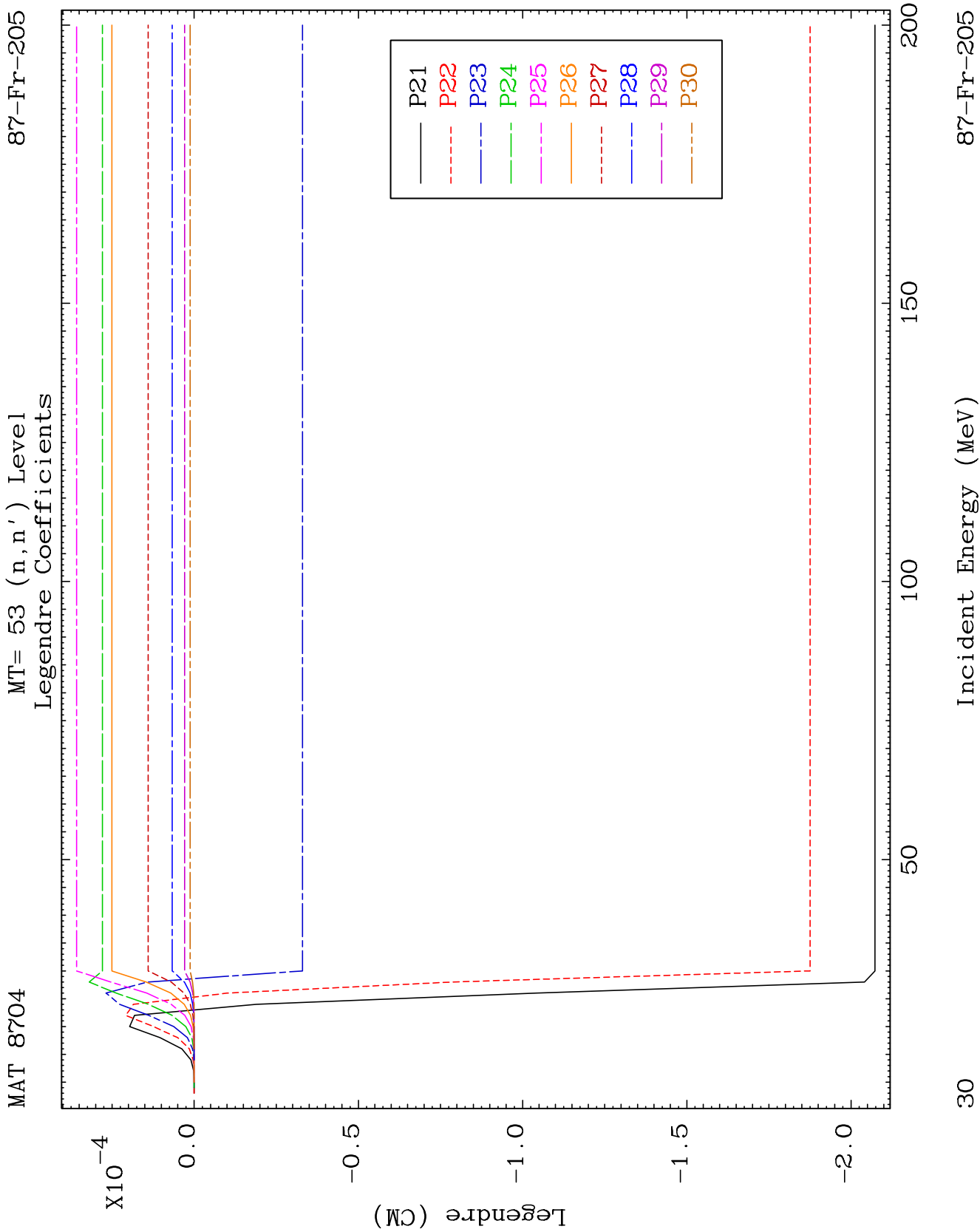


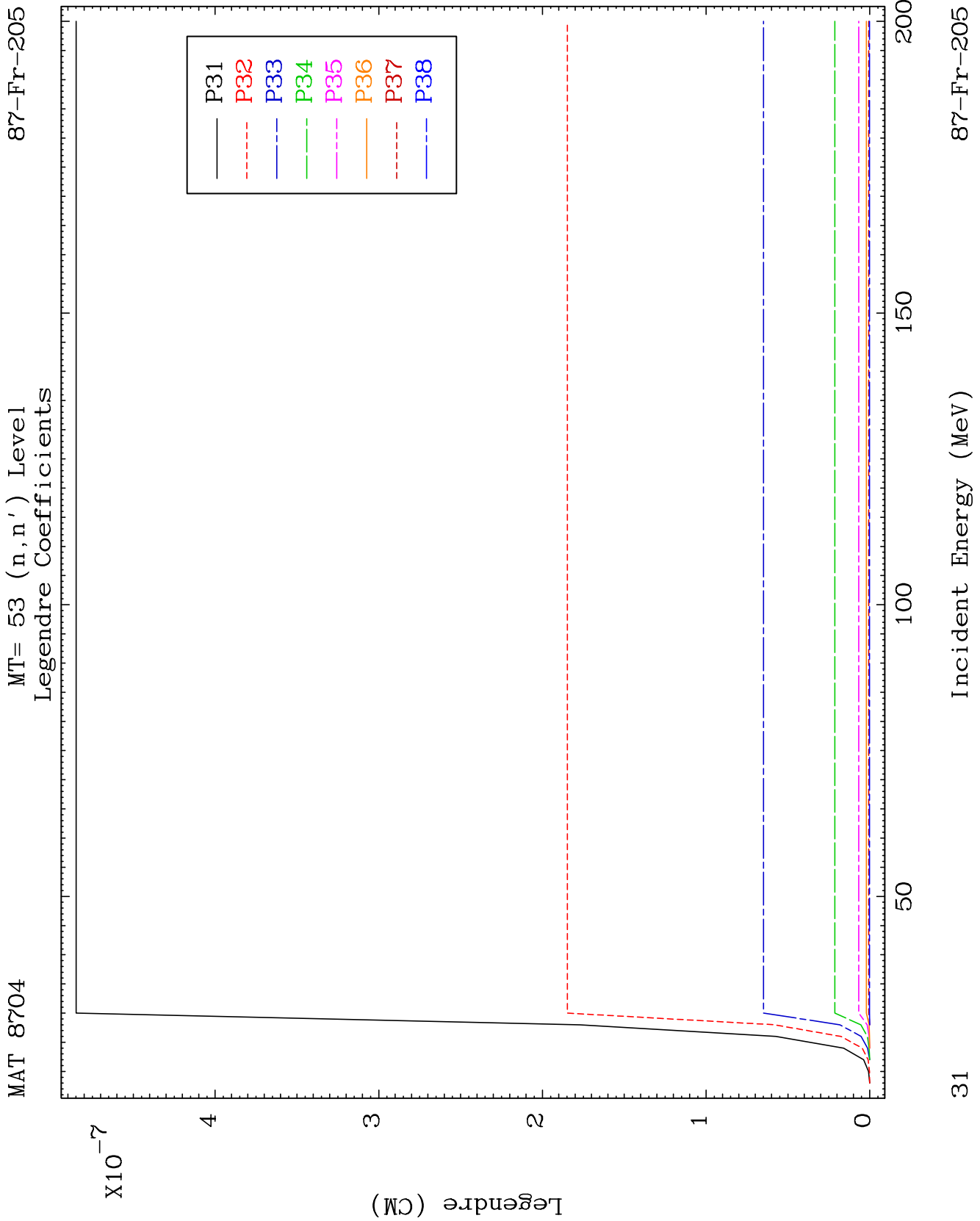
MAT 8704

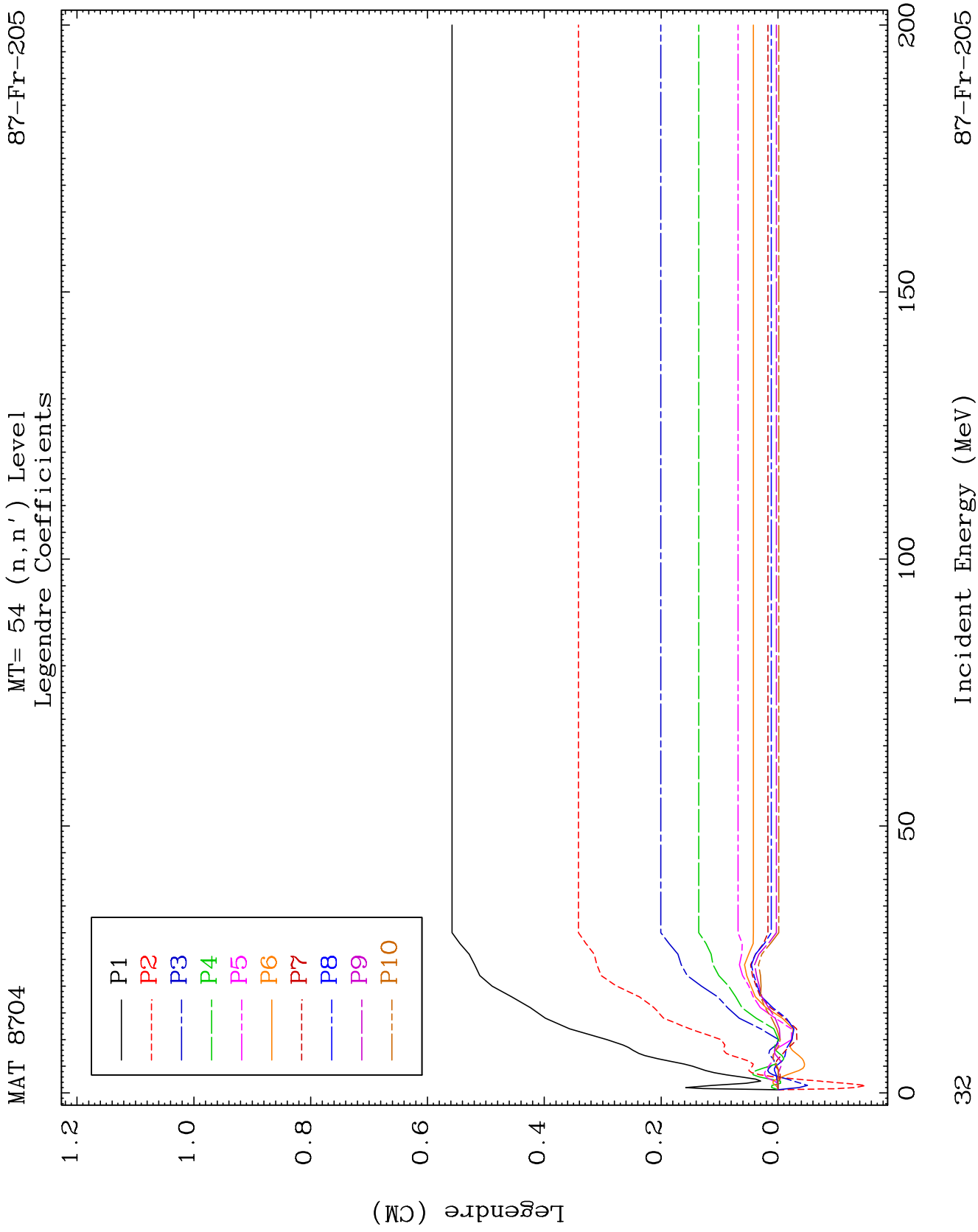
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Legendre Coefficients

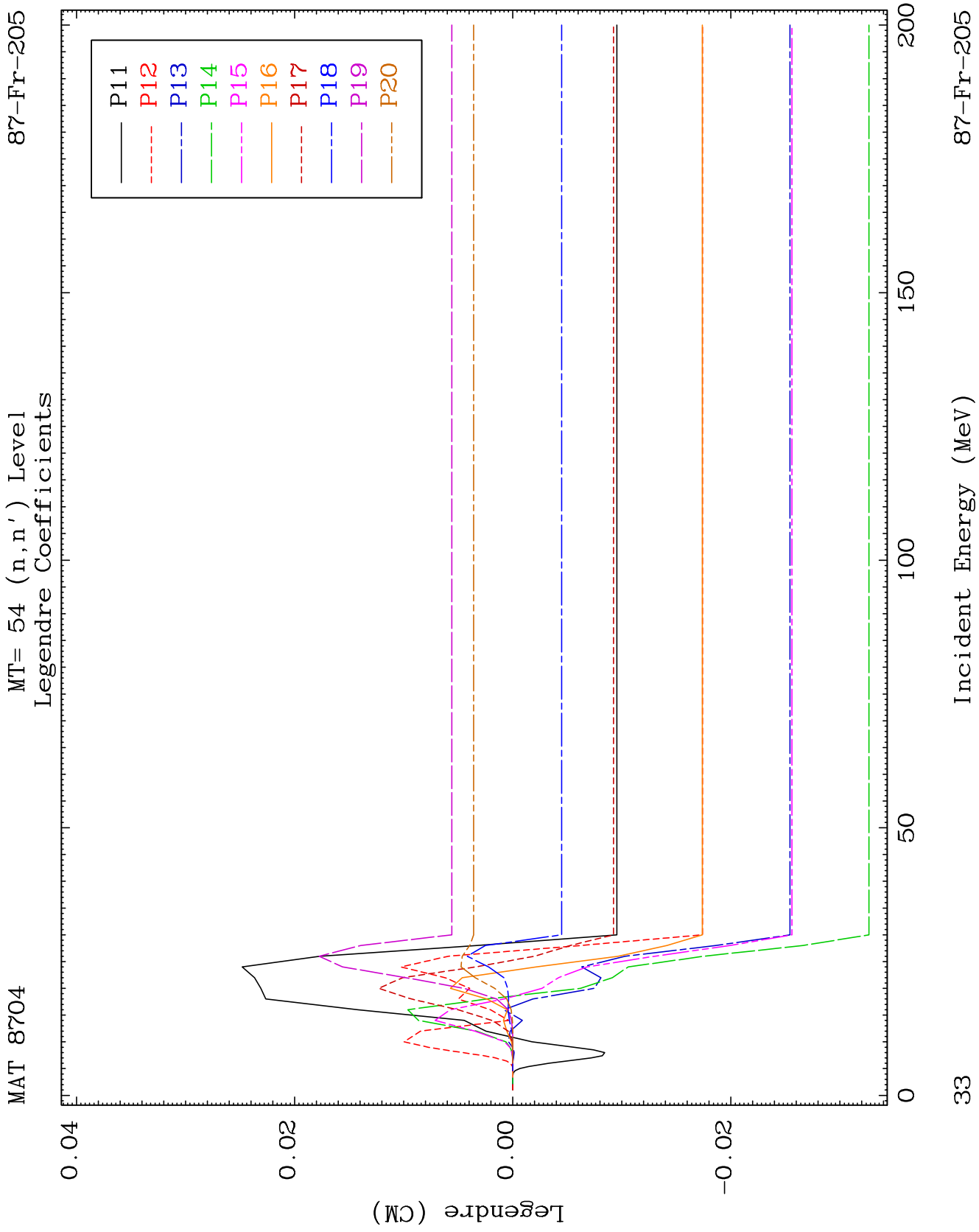
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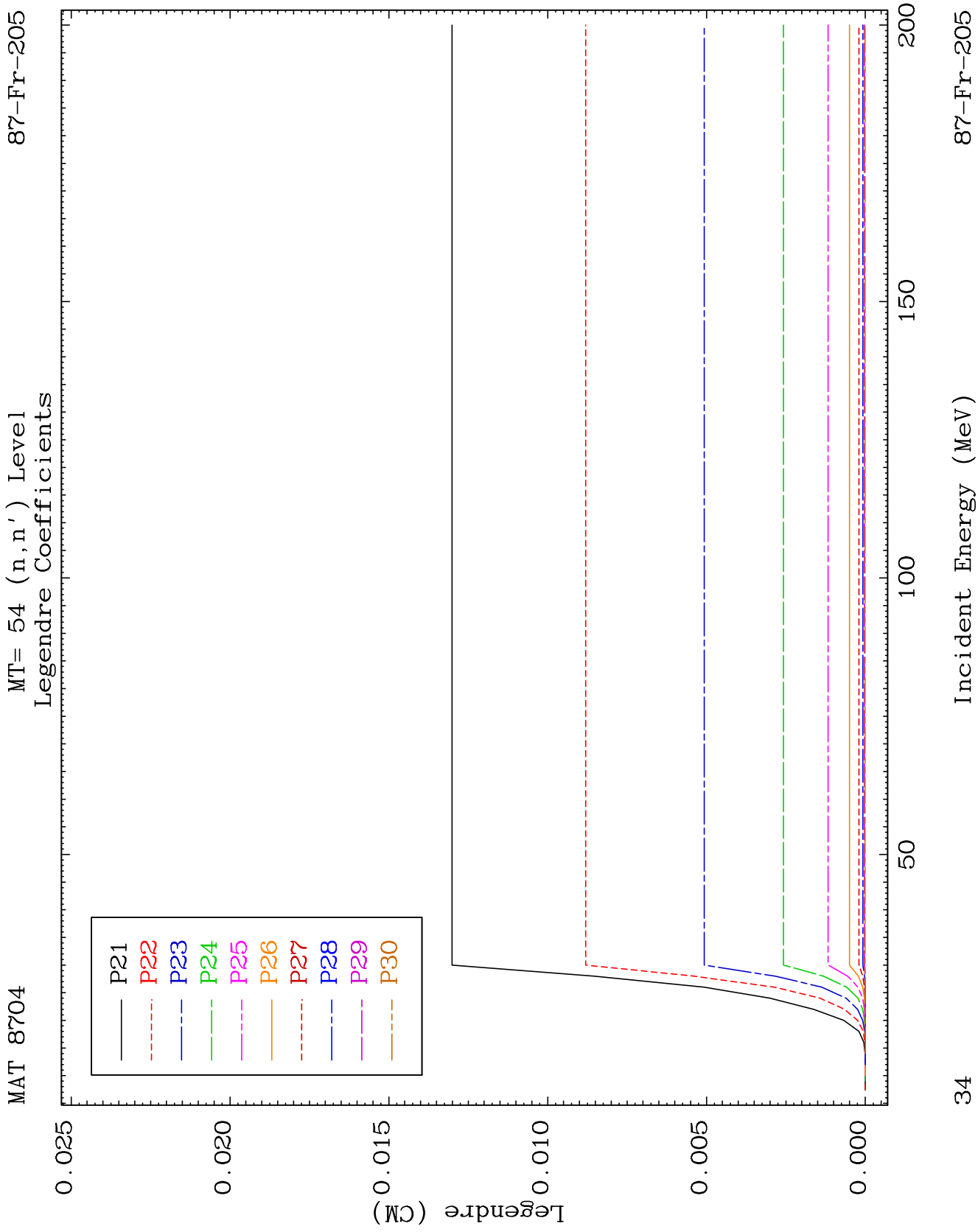








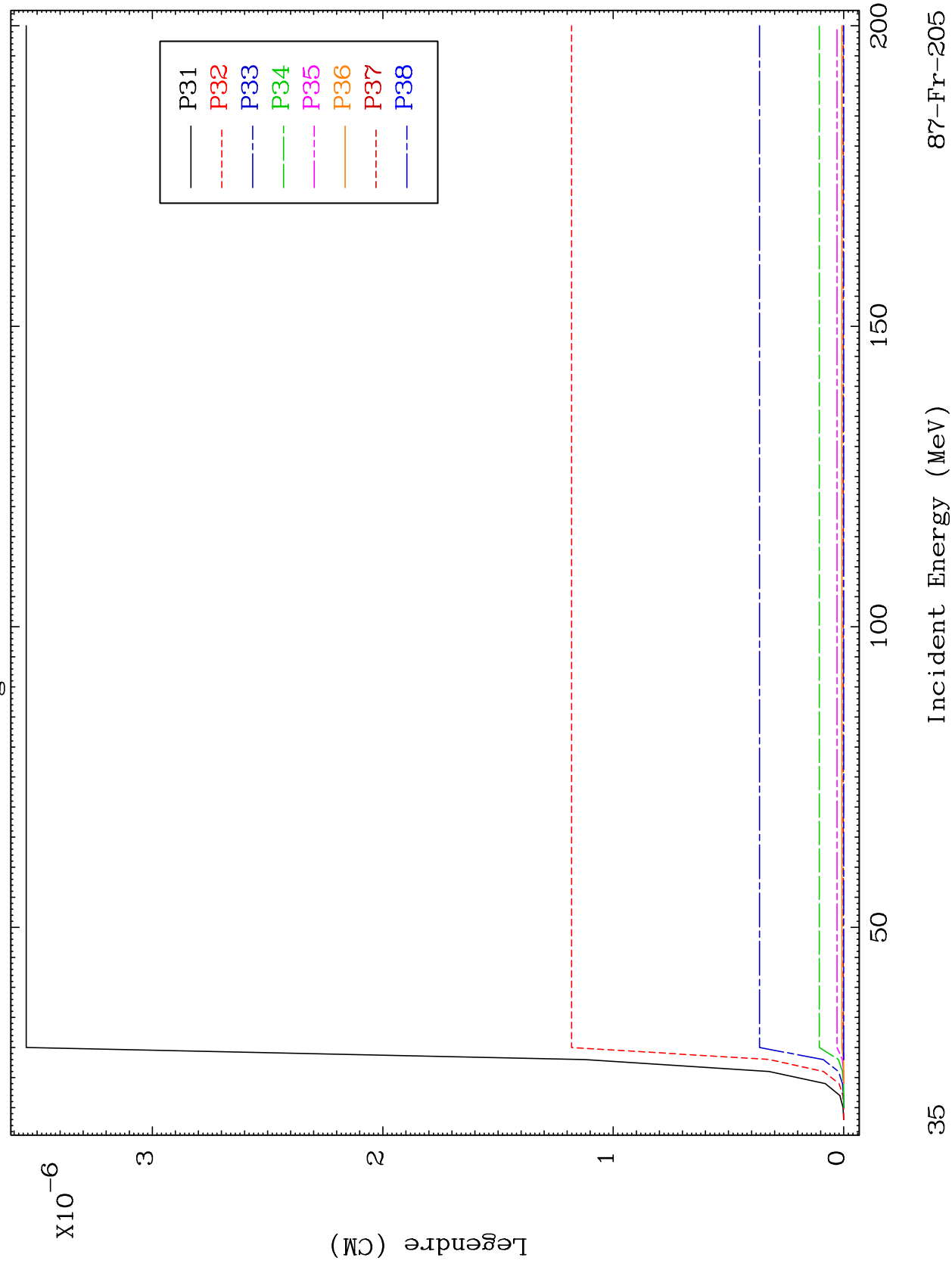


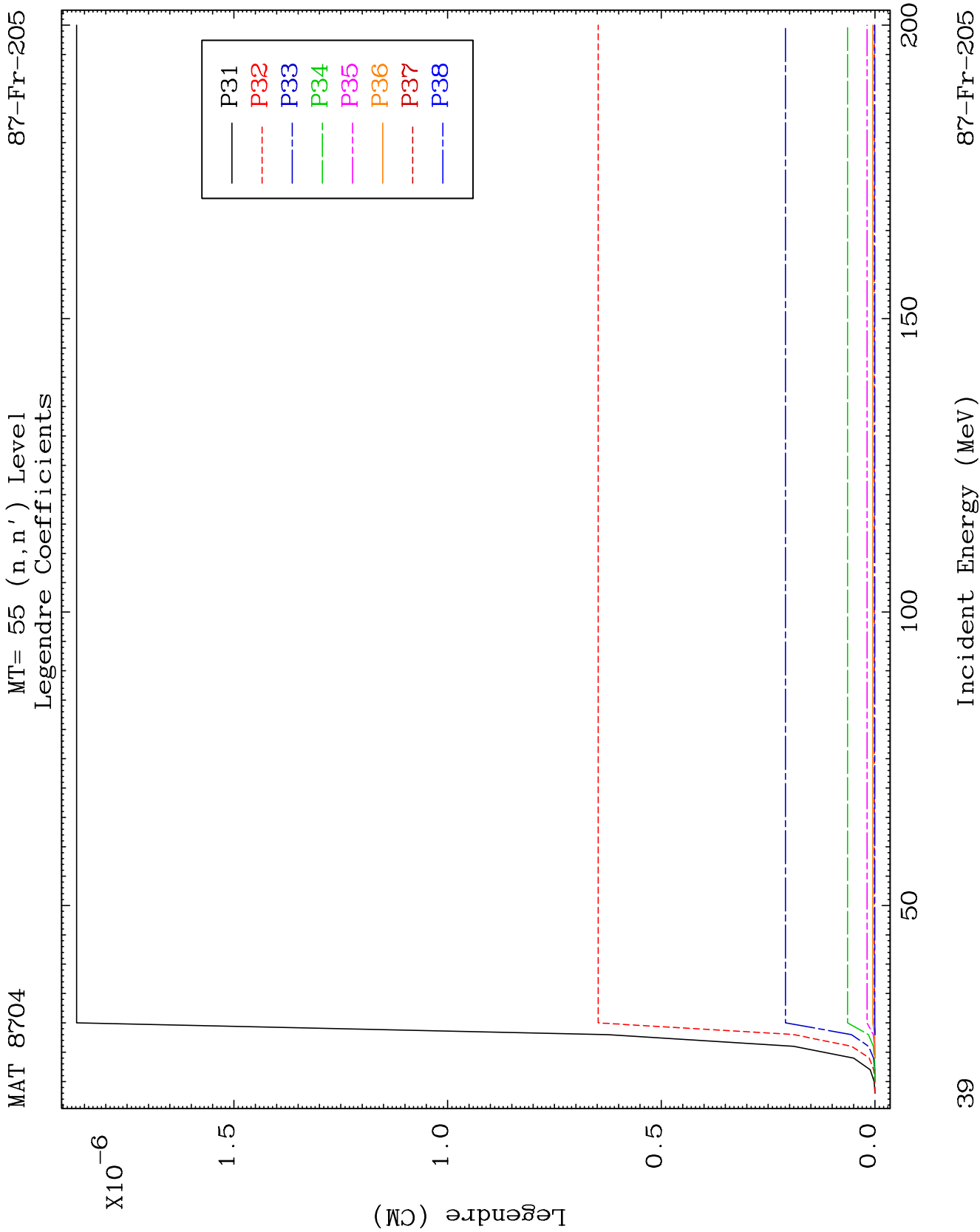


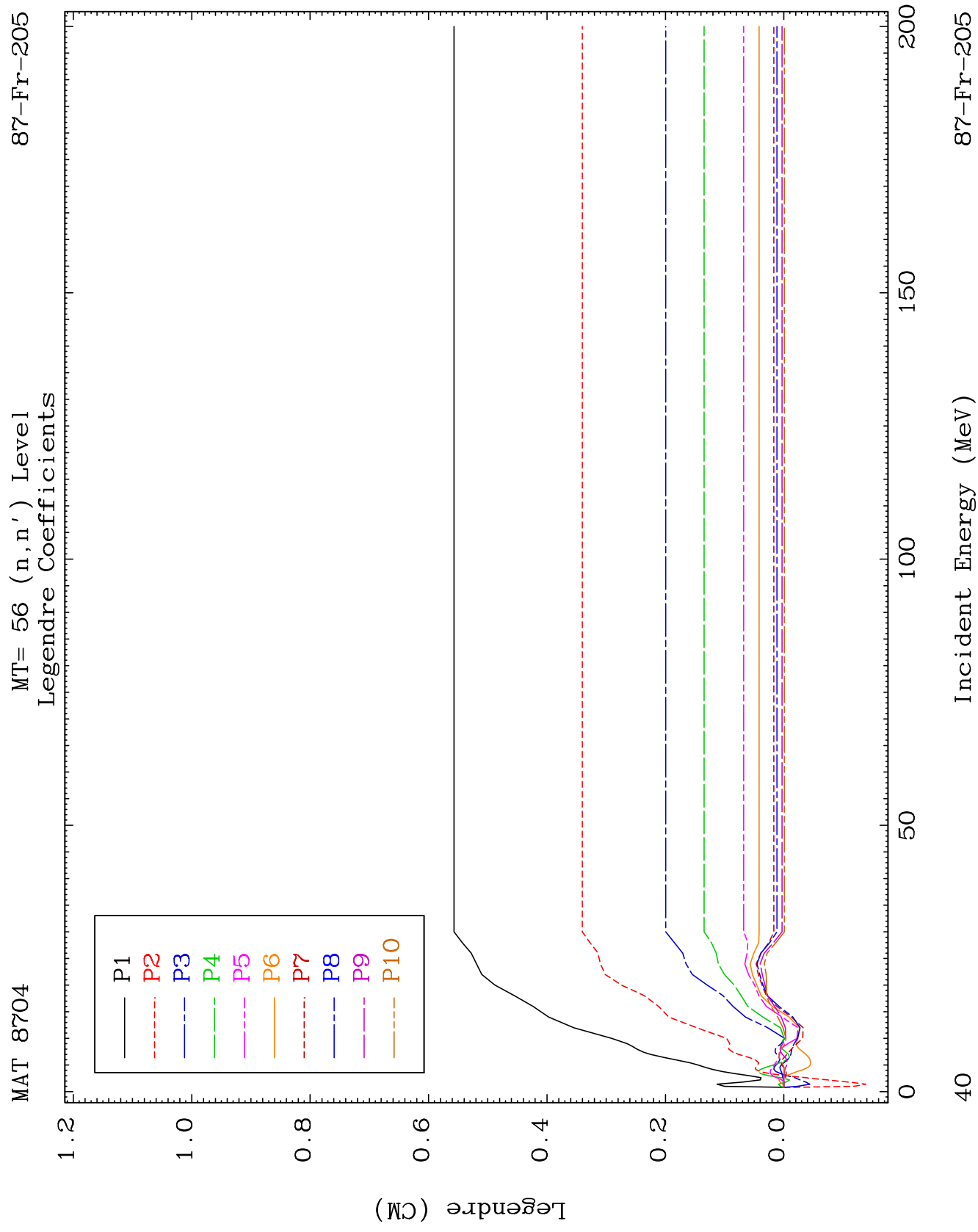
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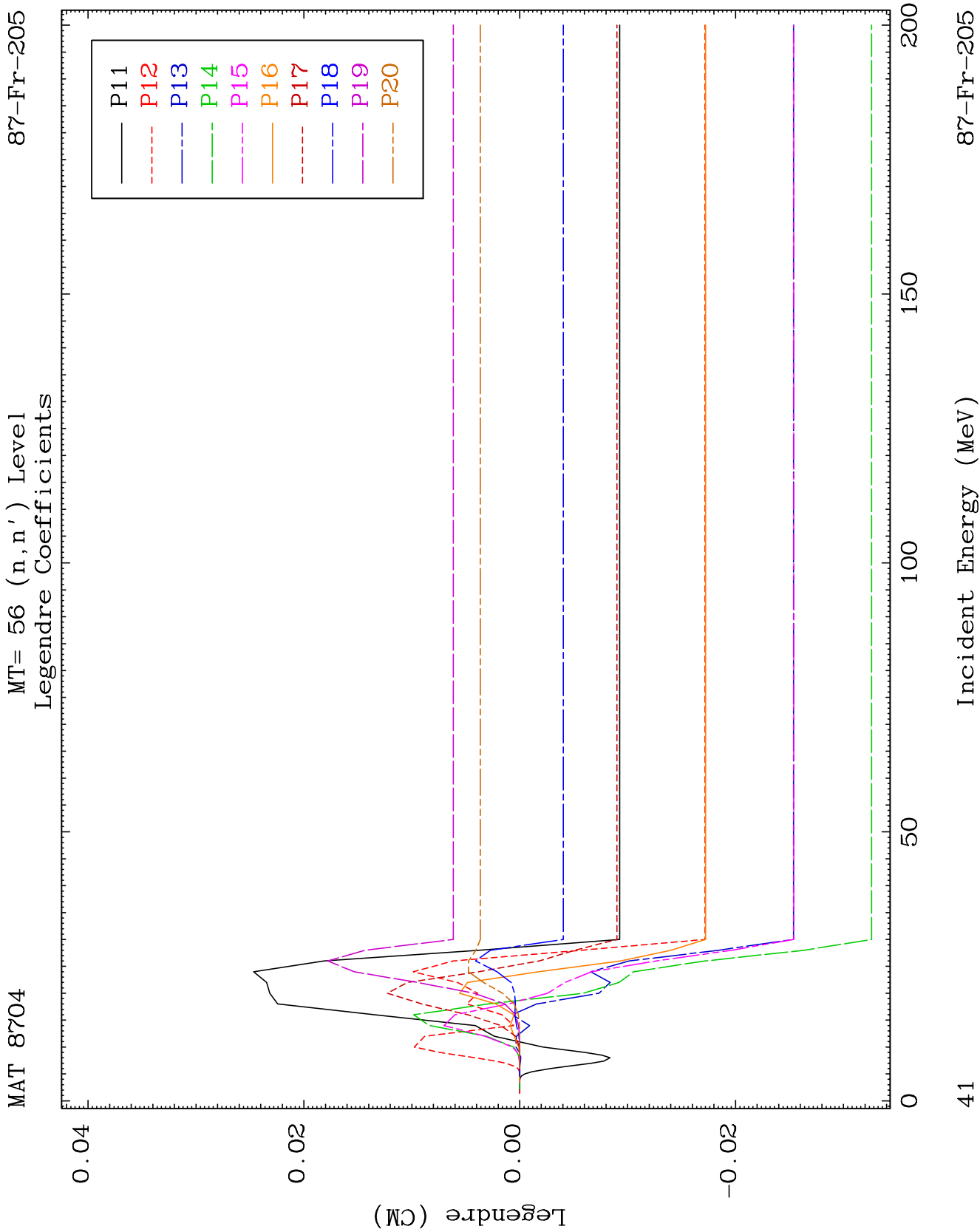
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Legendre Coefficients

87-Fr-205





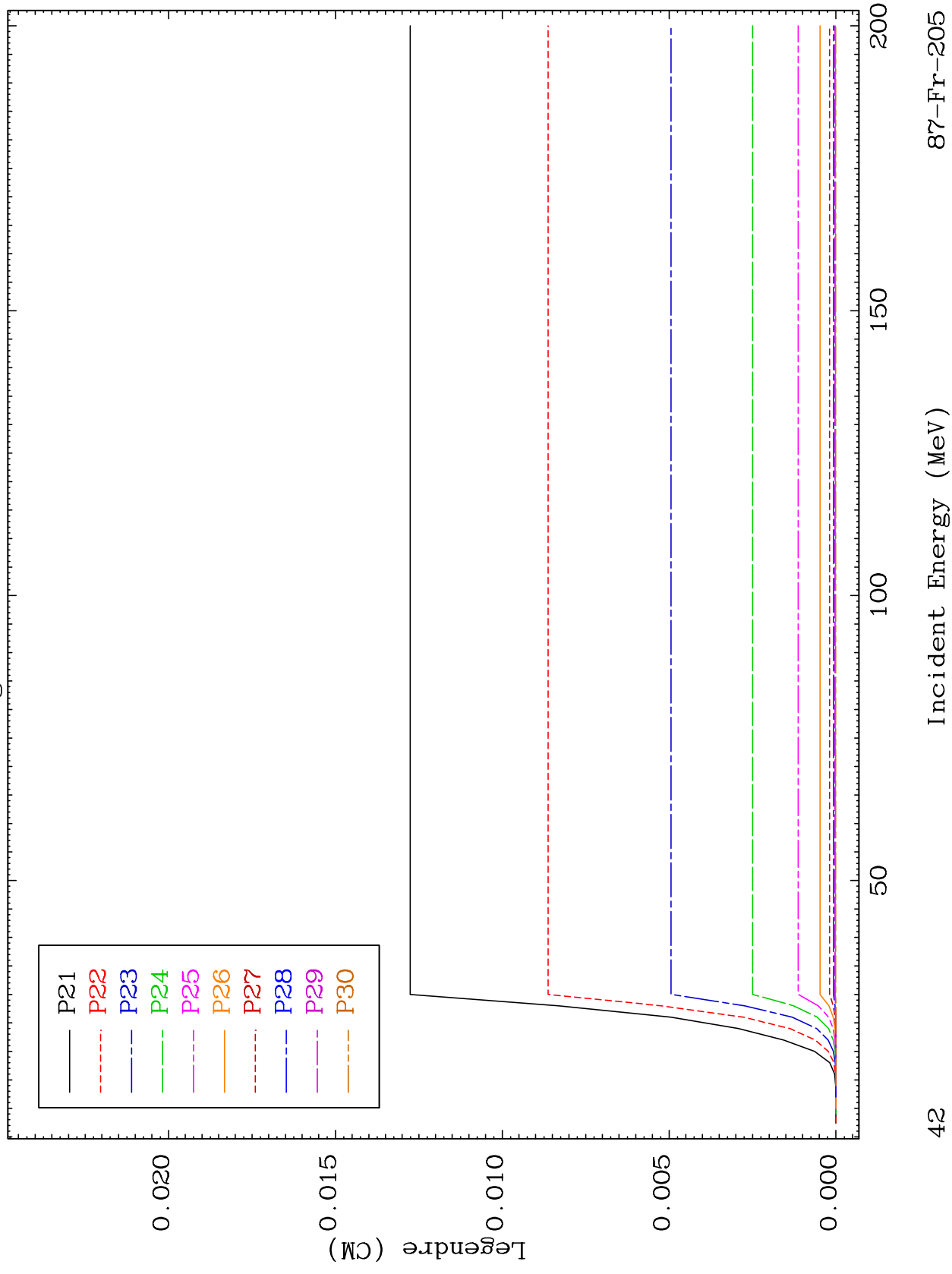


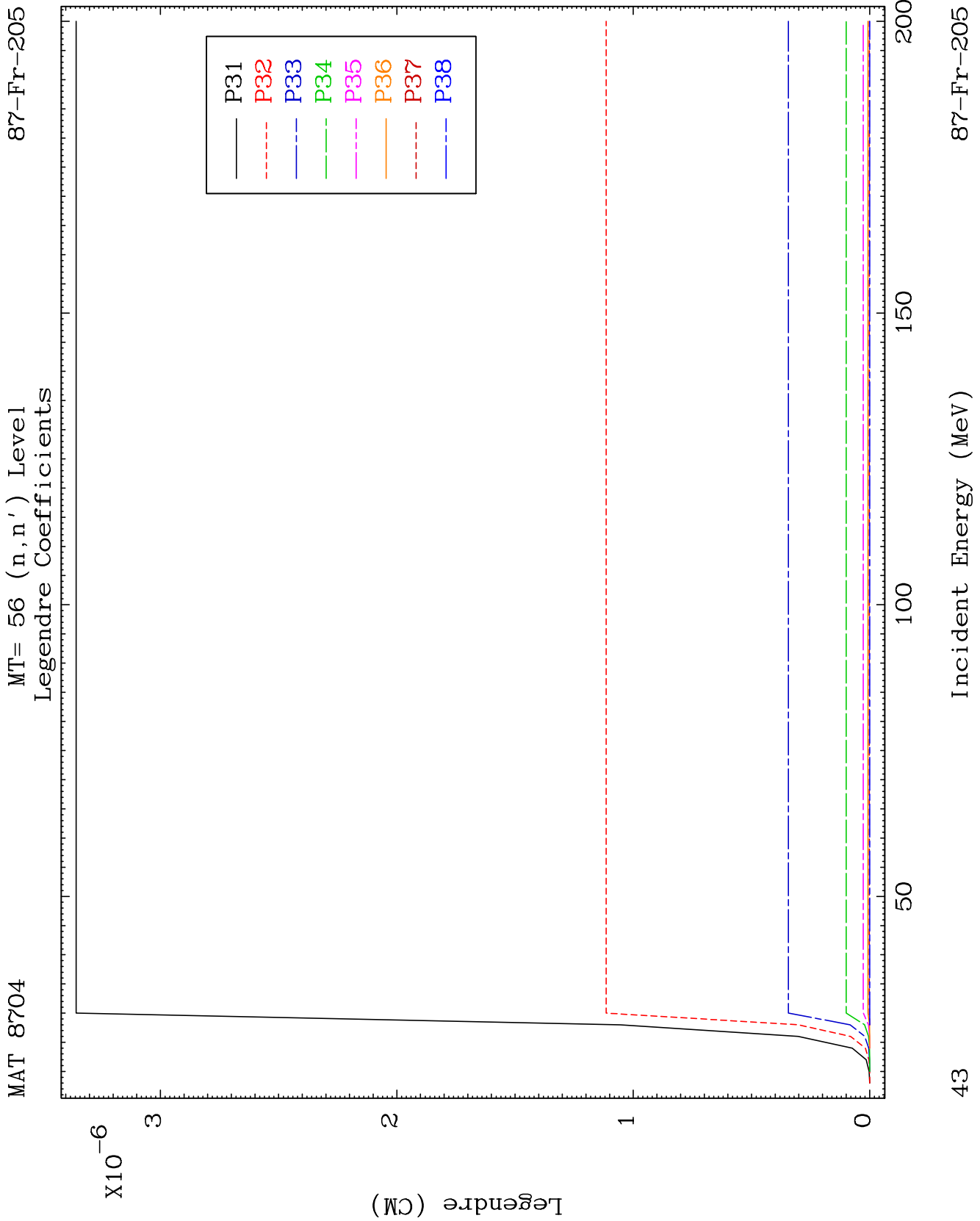


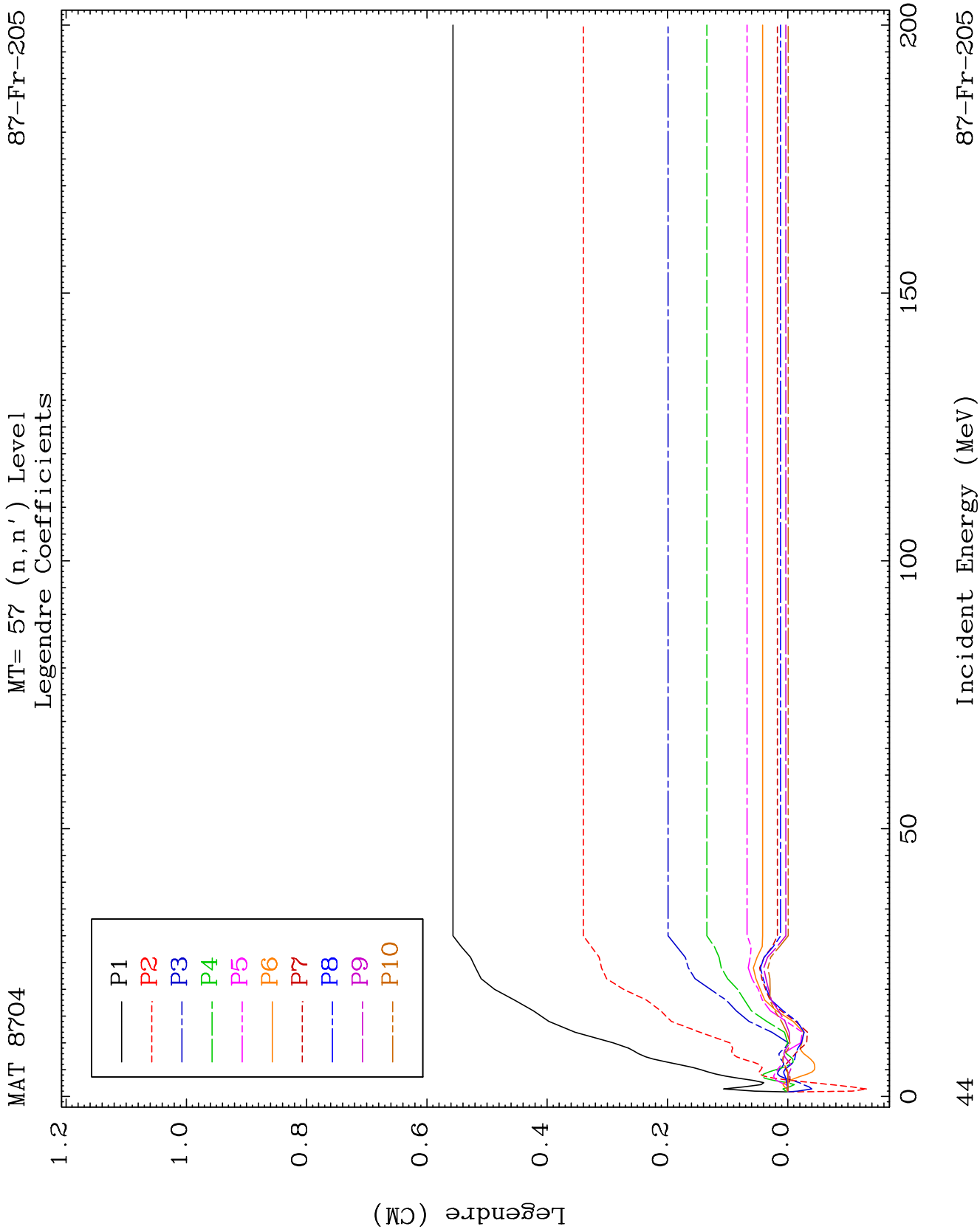
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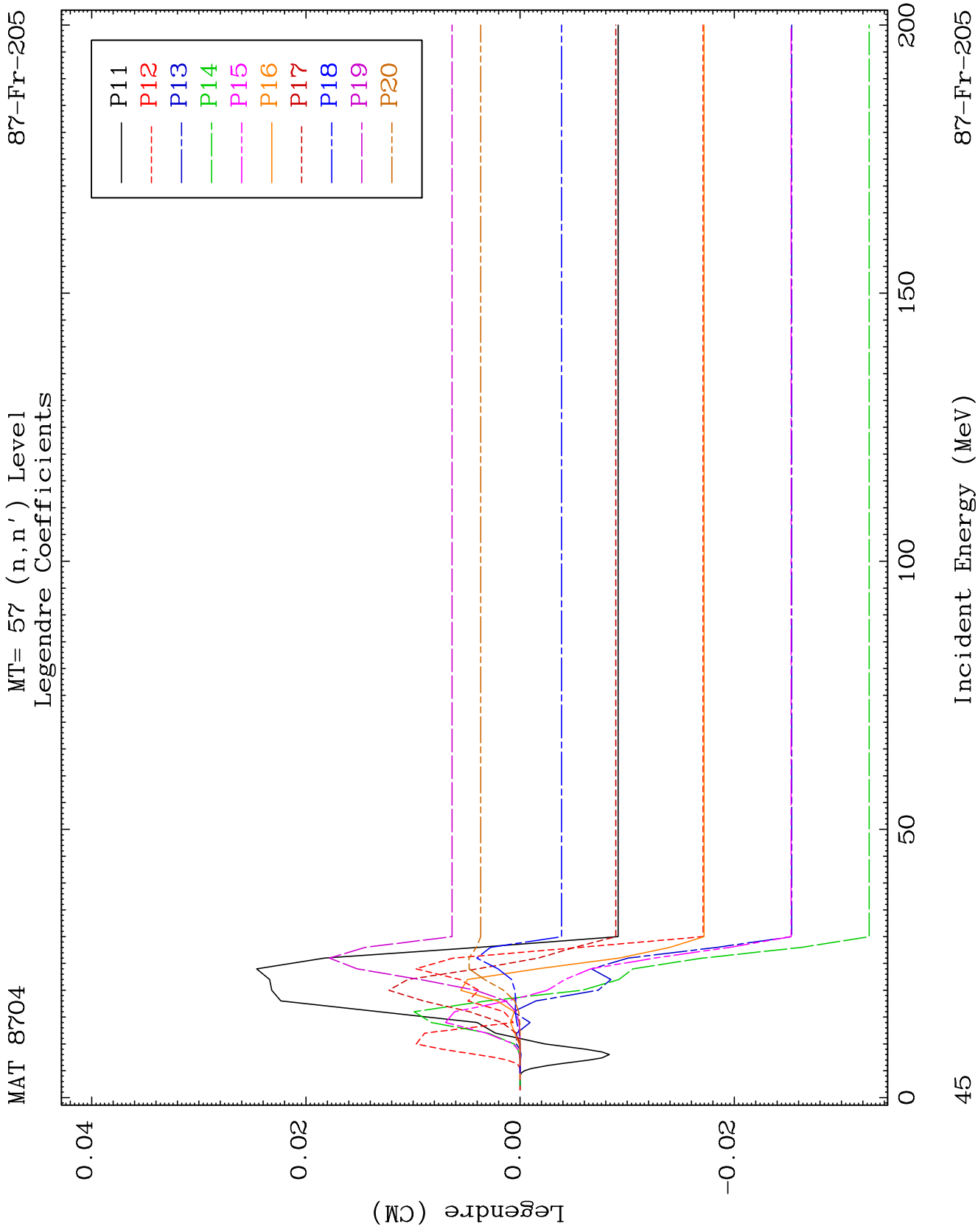
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Legendre Coefficients

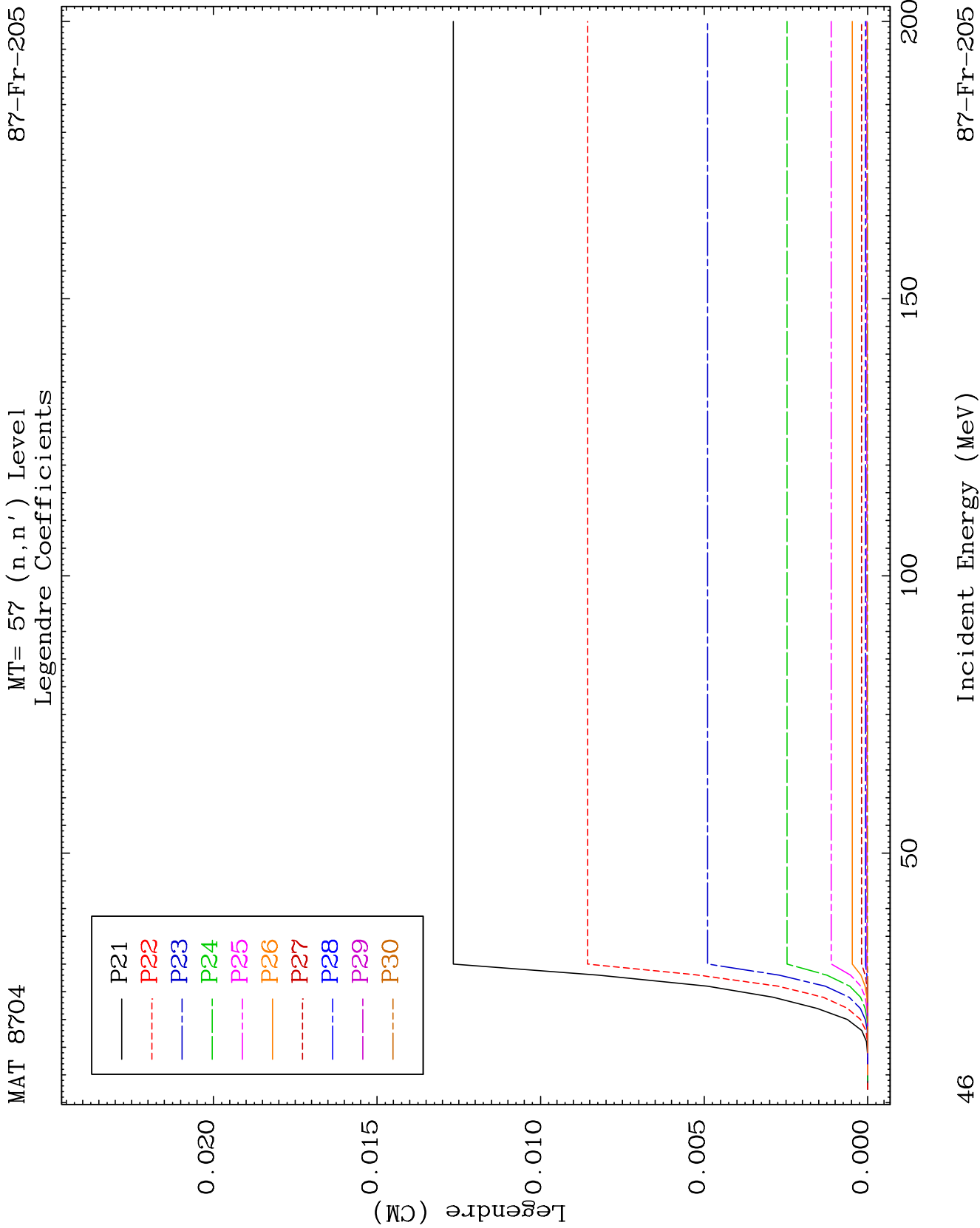
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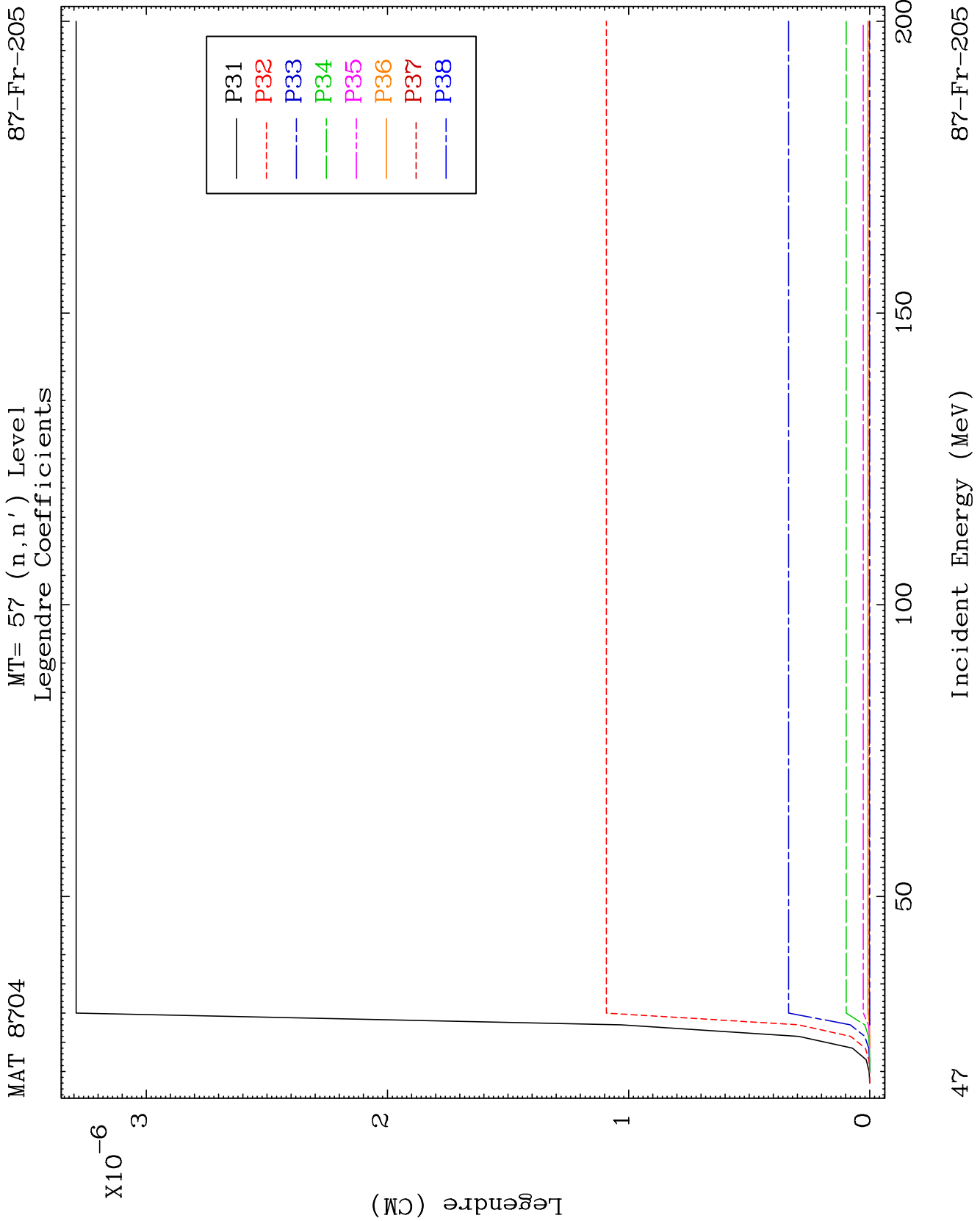








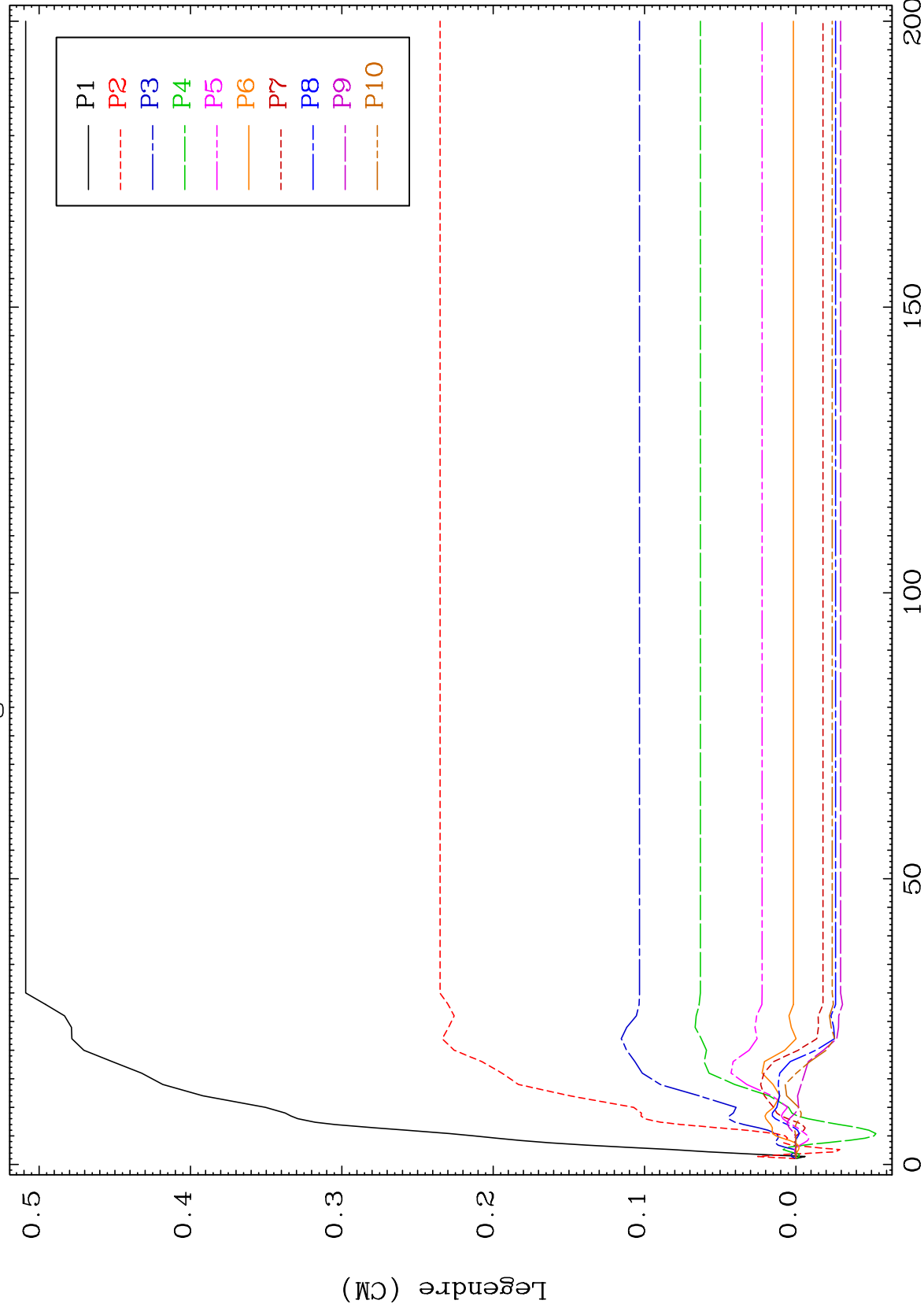




MAT 8704

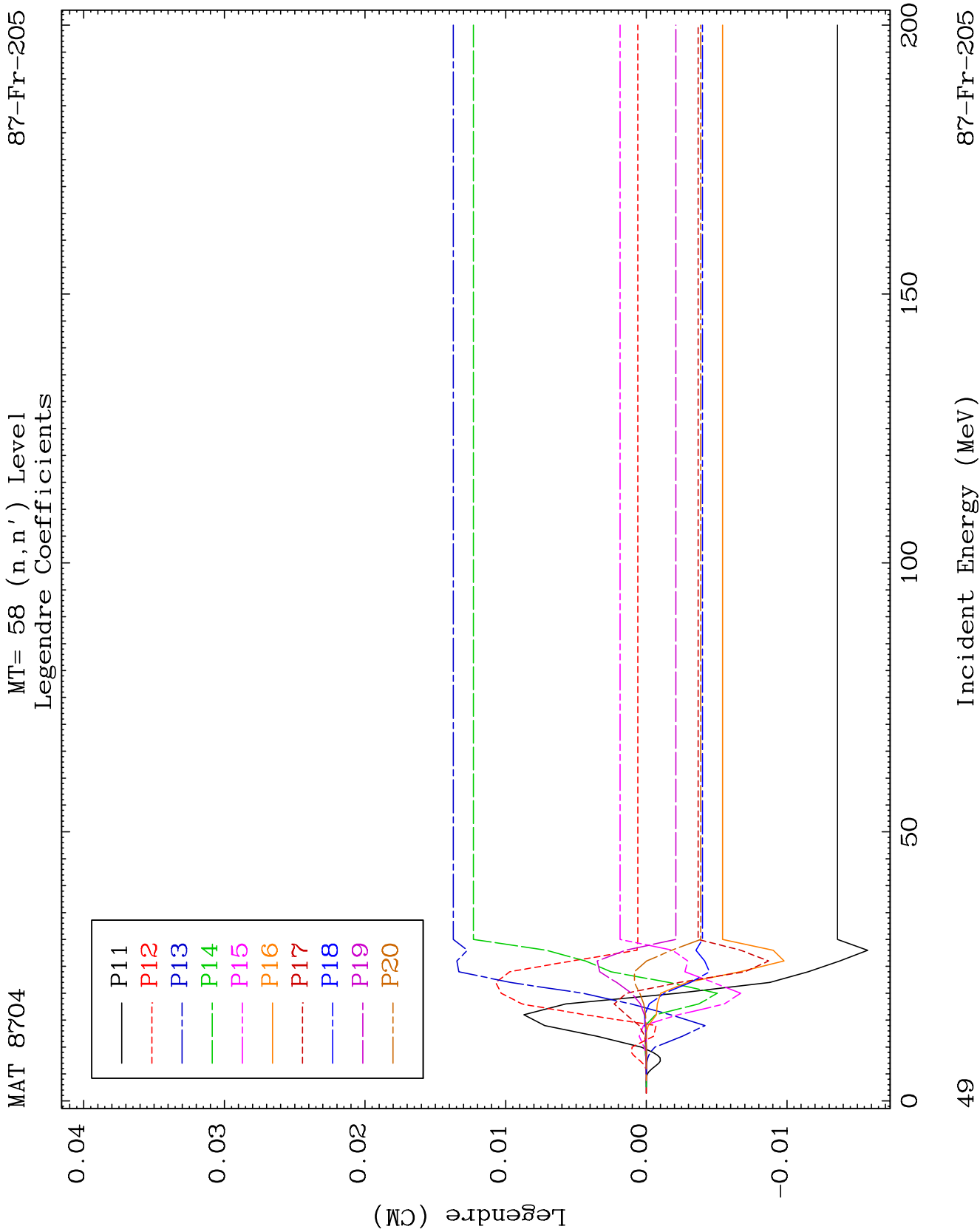
MT= 58 (n, n') Level
Legendre Coefficients

87-Fr-205



87-Fr-205

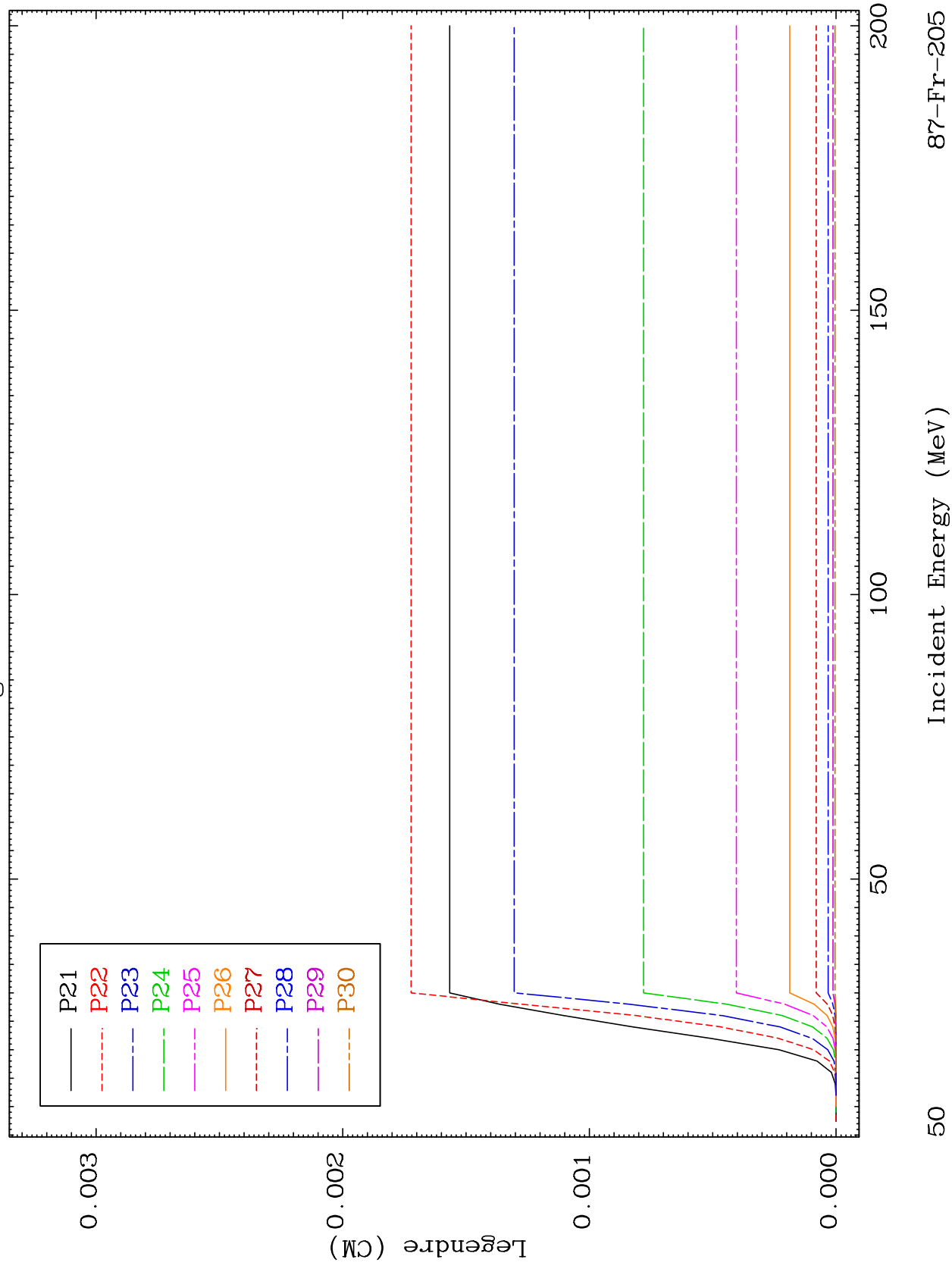
Incident Energy (MeV)

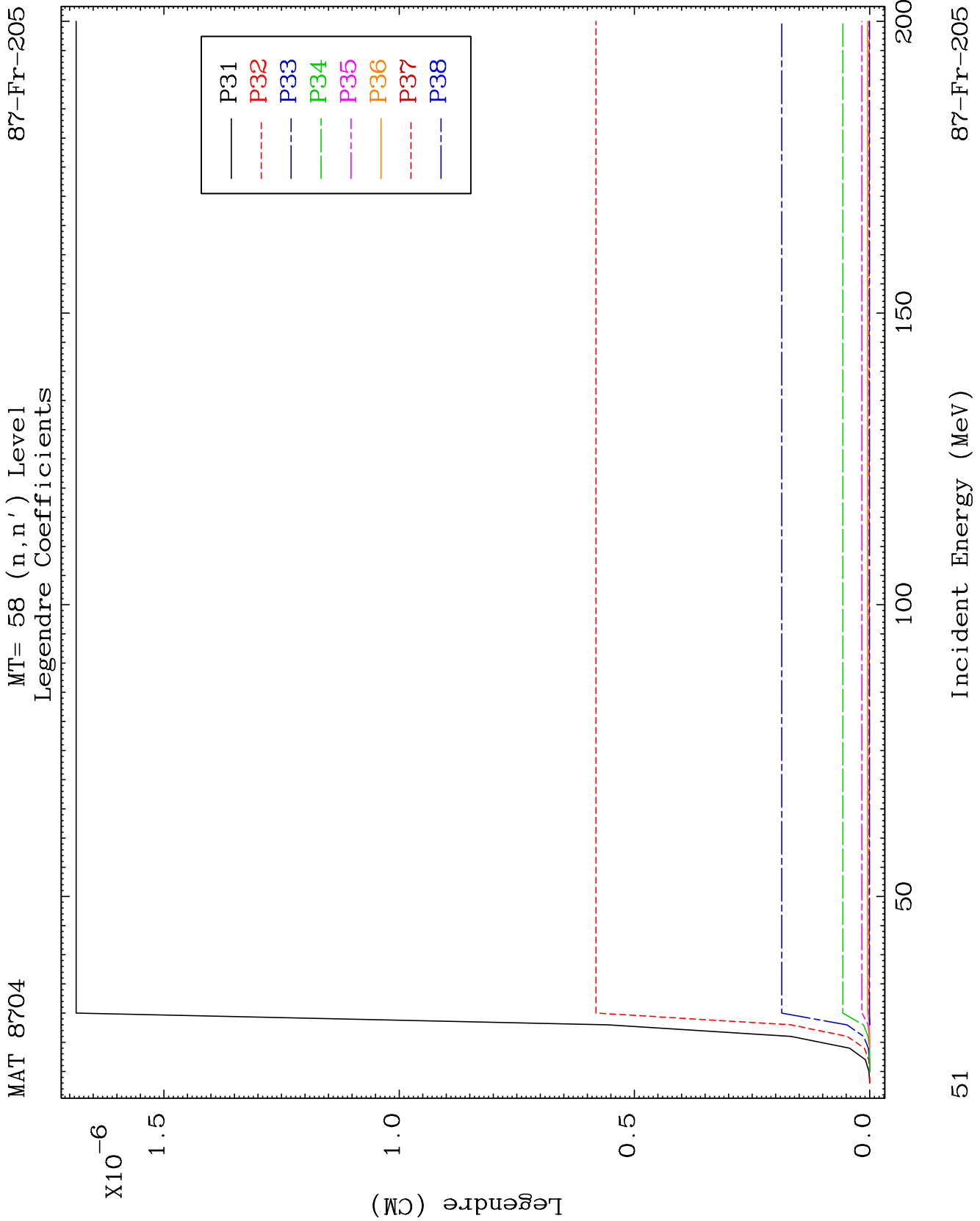


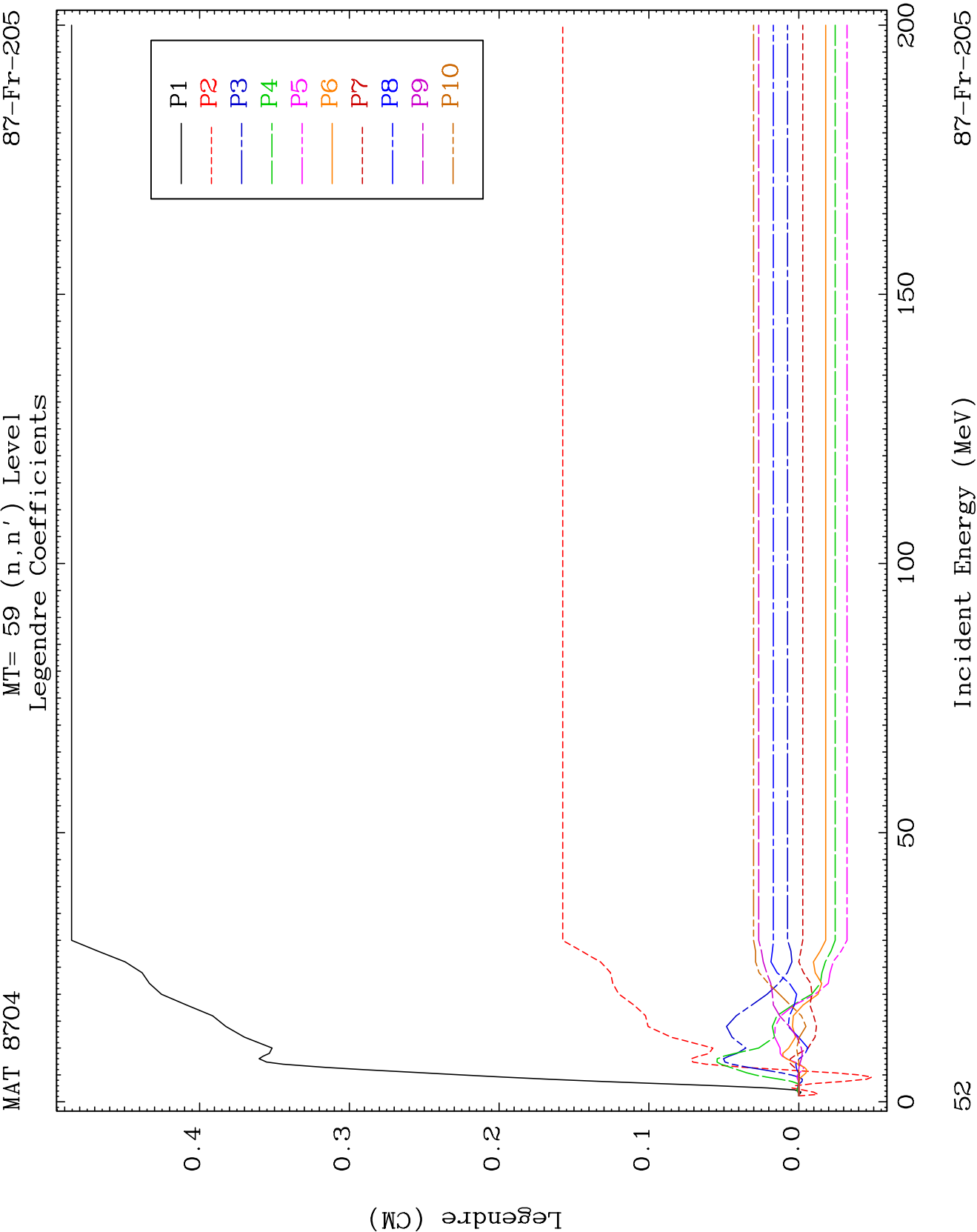
MAT 8704

MT= 58 (n, n') Level
Legendre Coefficients

87-Fr-205



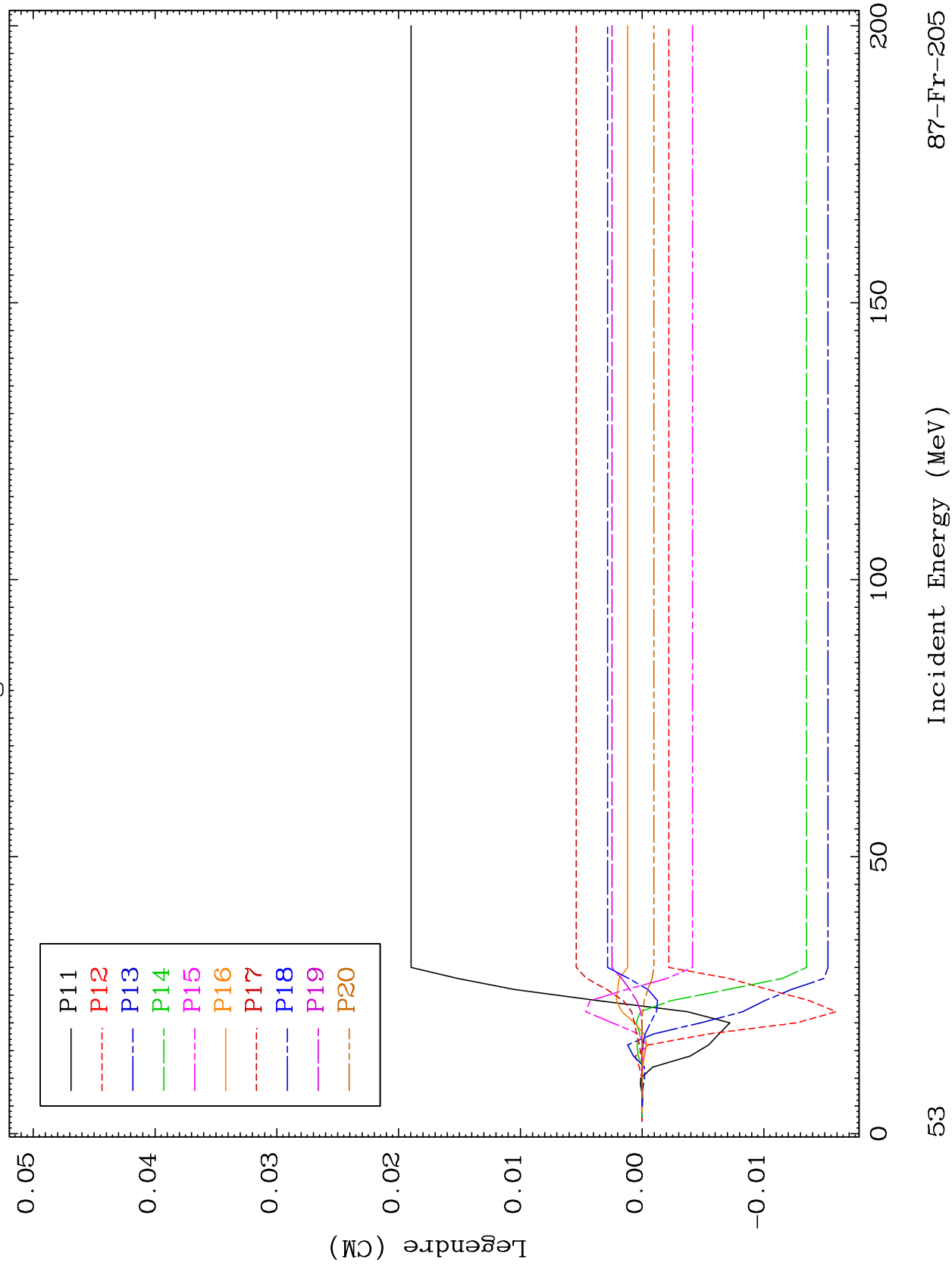


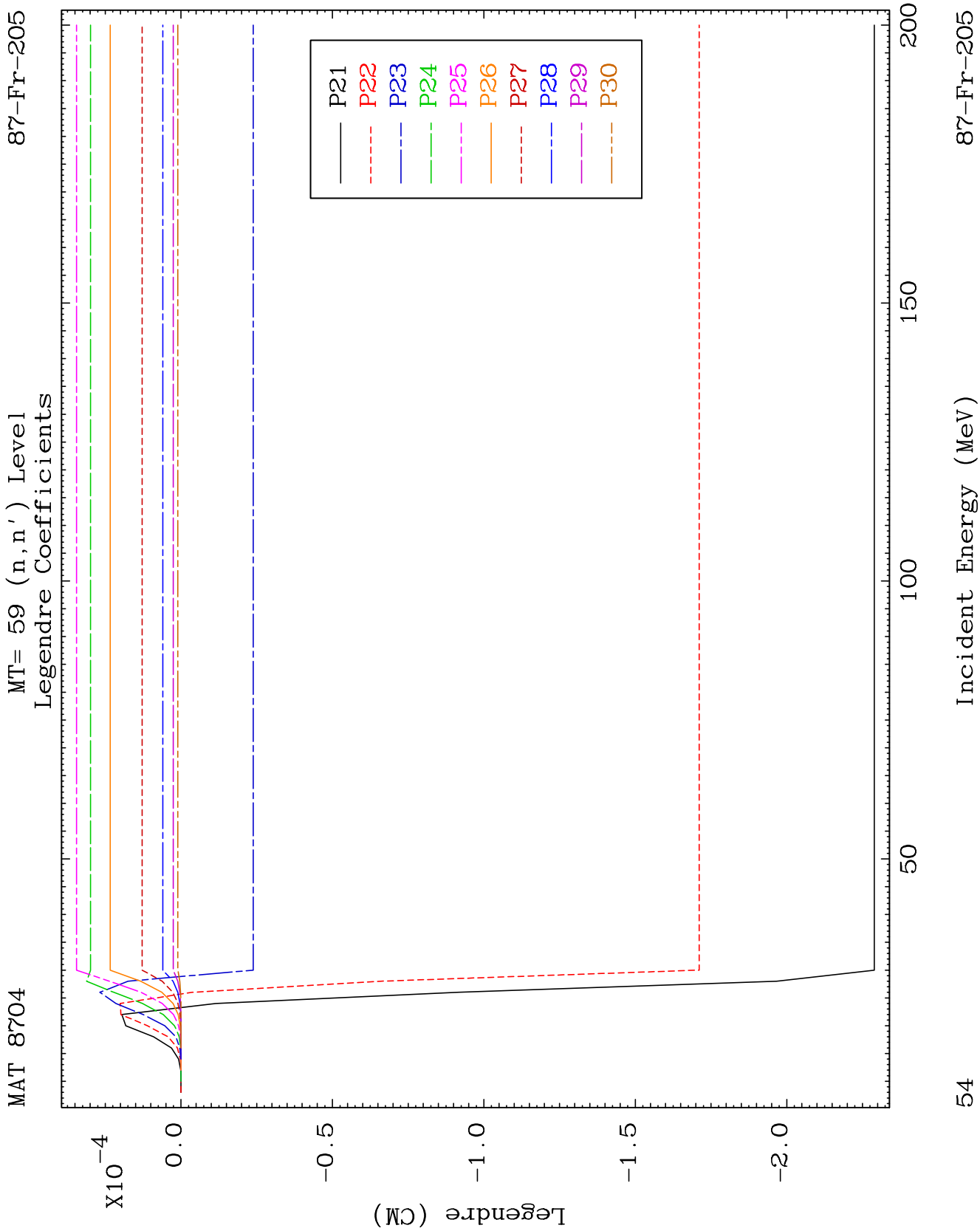


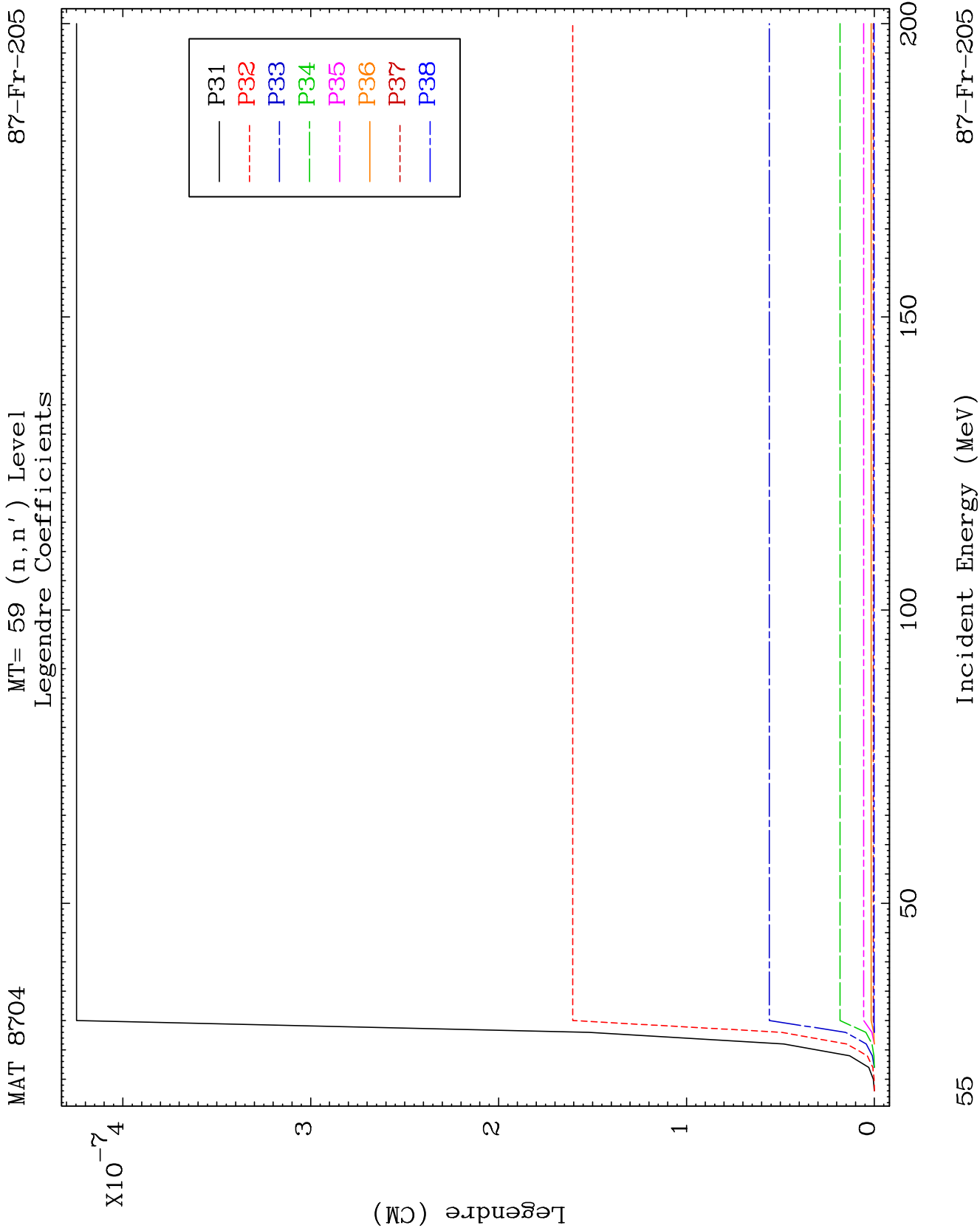
MAT 8704

MT= 59 (n, n') Level
Legendre Coefficients

87-Fr-205



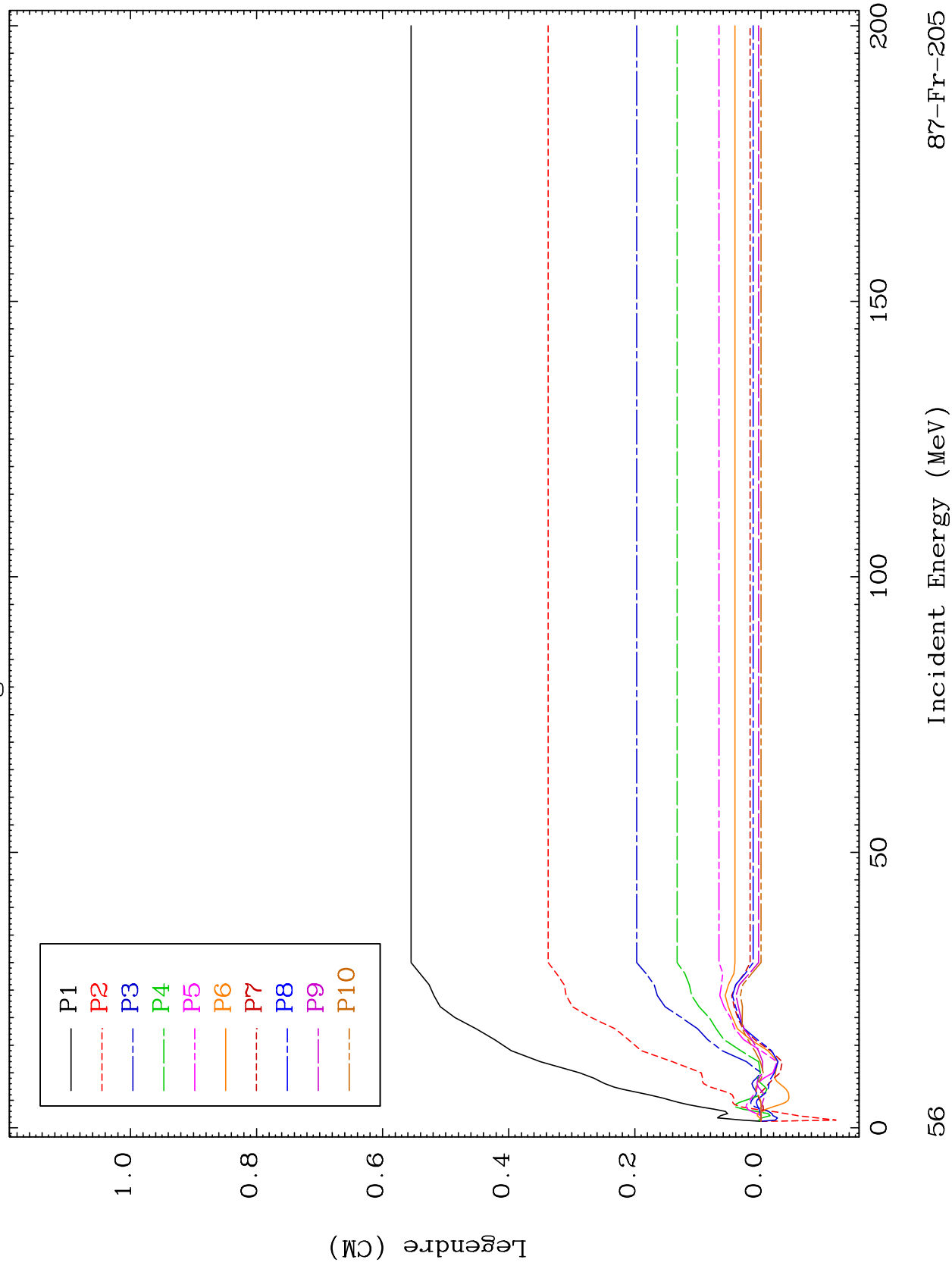


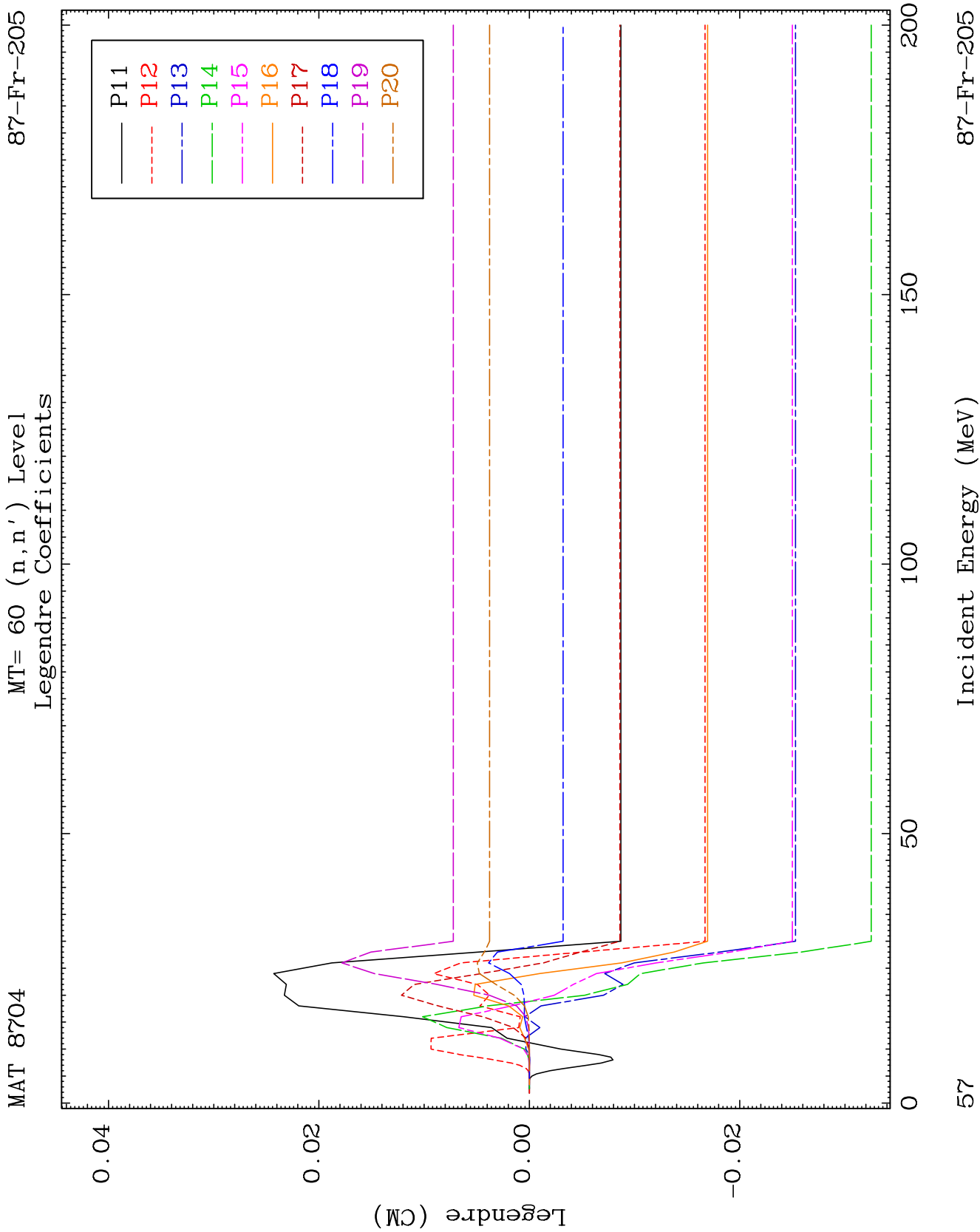


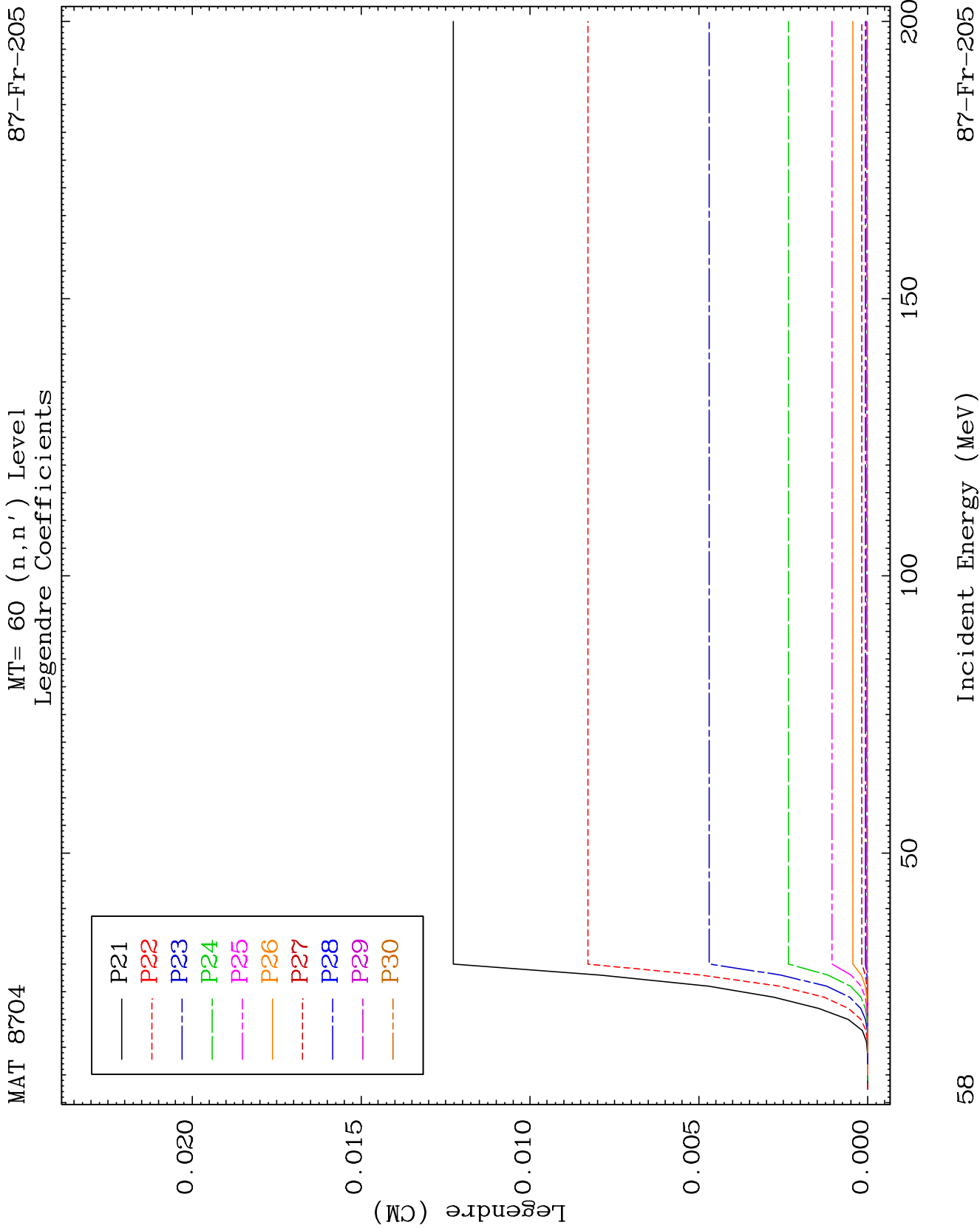
MAT 8704

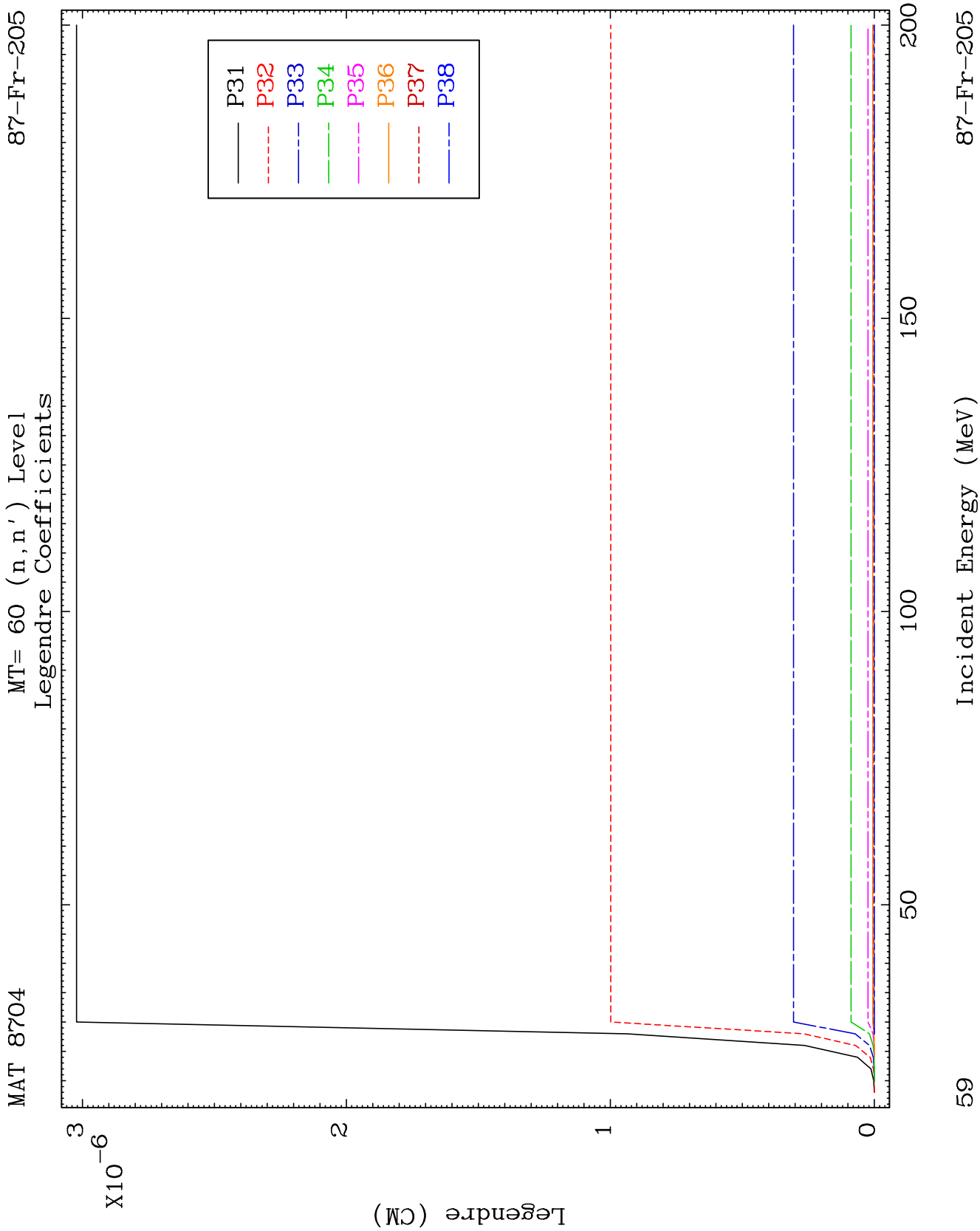
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Legendre Coefficients

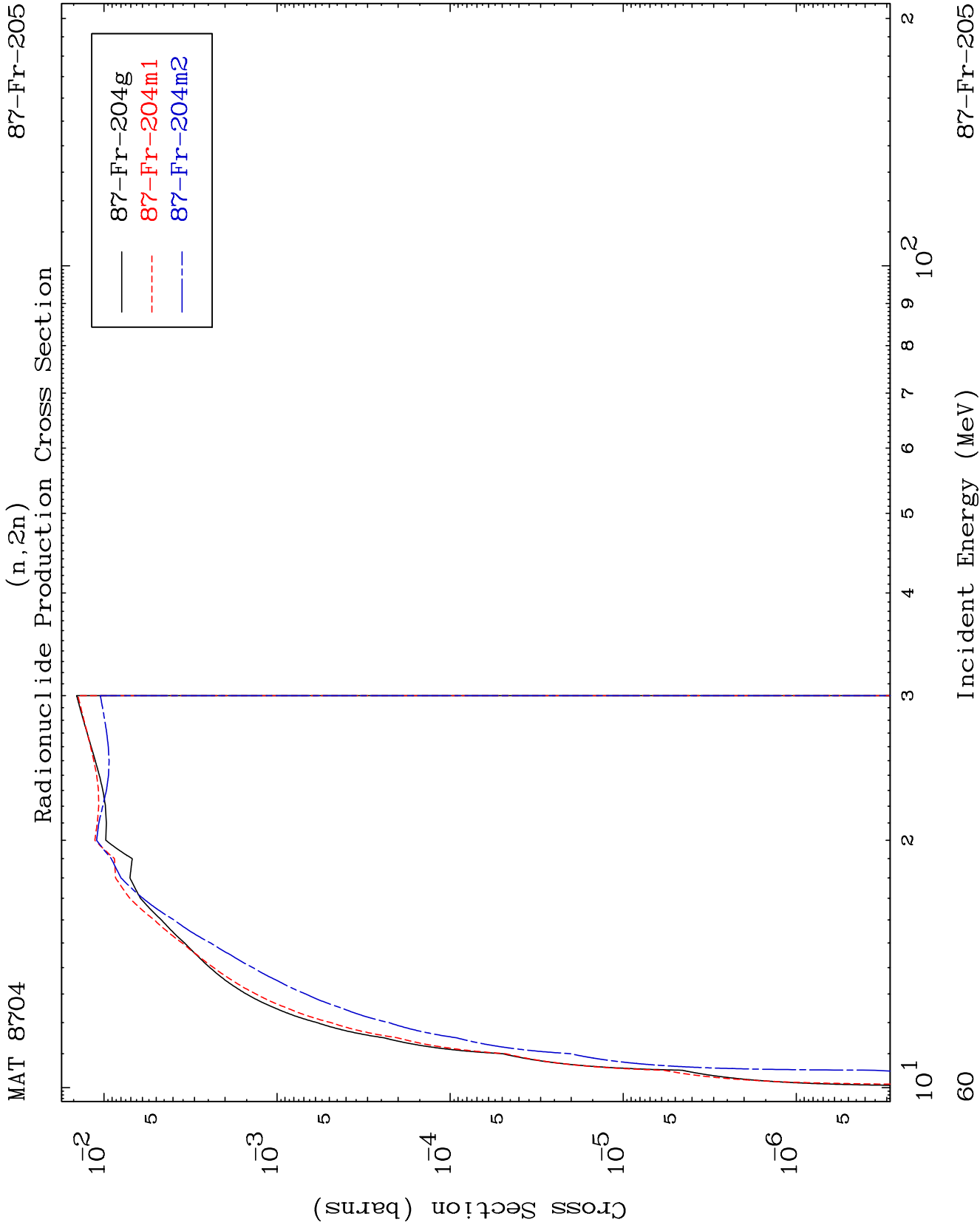
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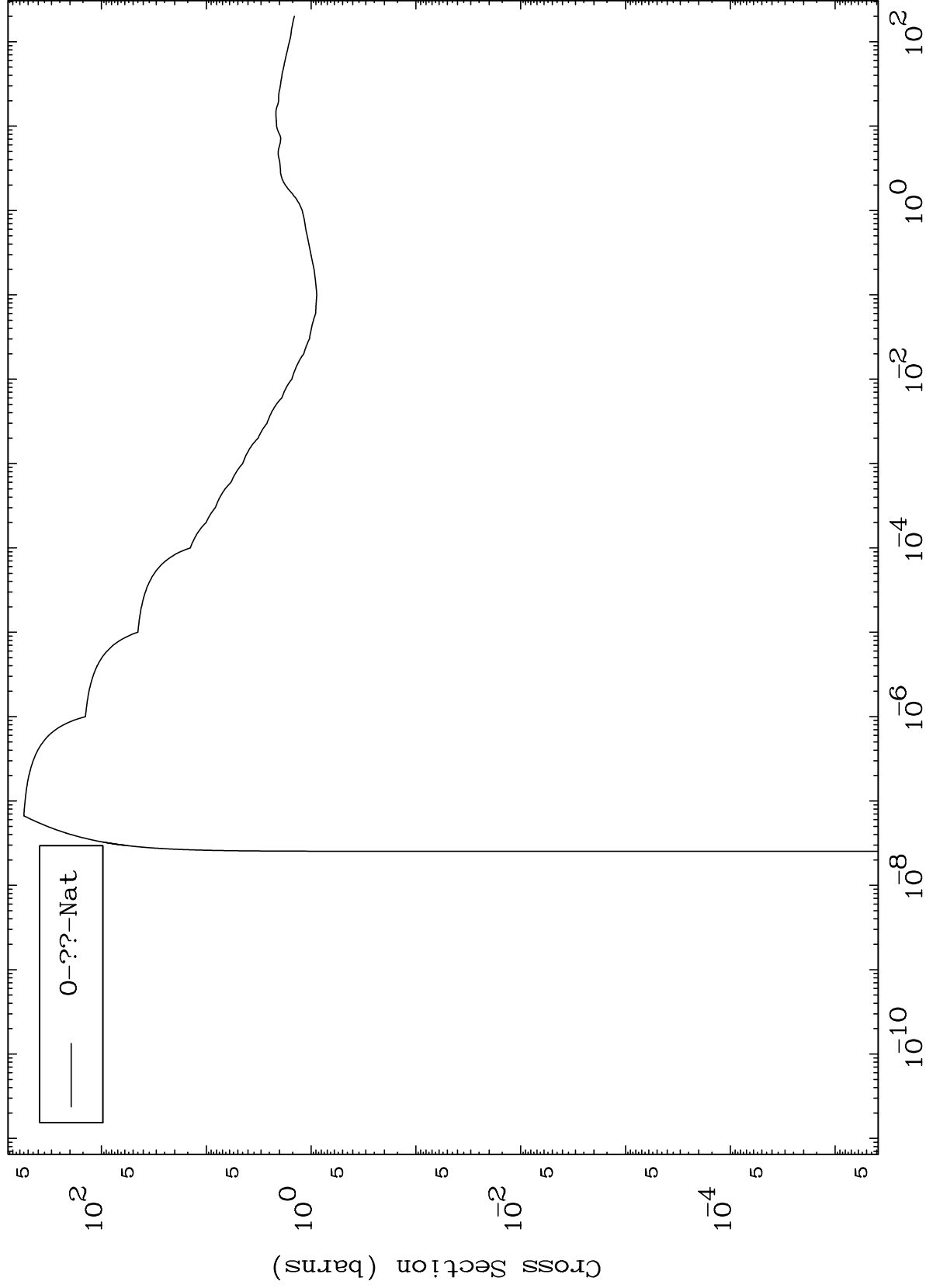


MAT 8704

Fission

87-Fr-205

Radionuclide Production Cross Section



61

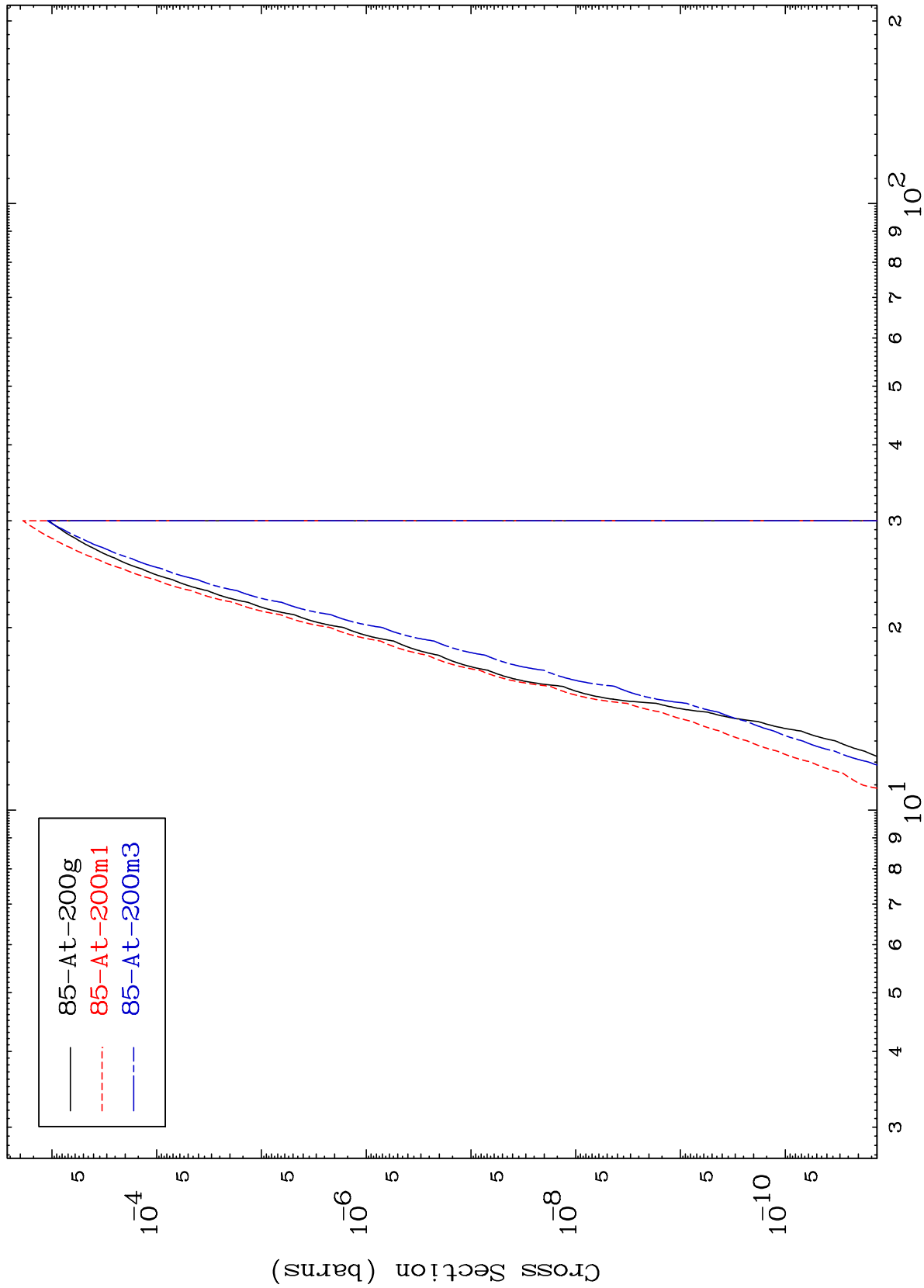
Incident Energy (MeV)

87-Fr-205

MAT 8704

87-Fr-205

(n,2n) α
Radionuclide Production Cross Section



62

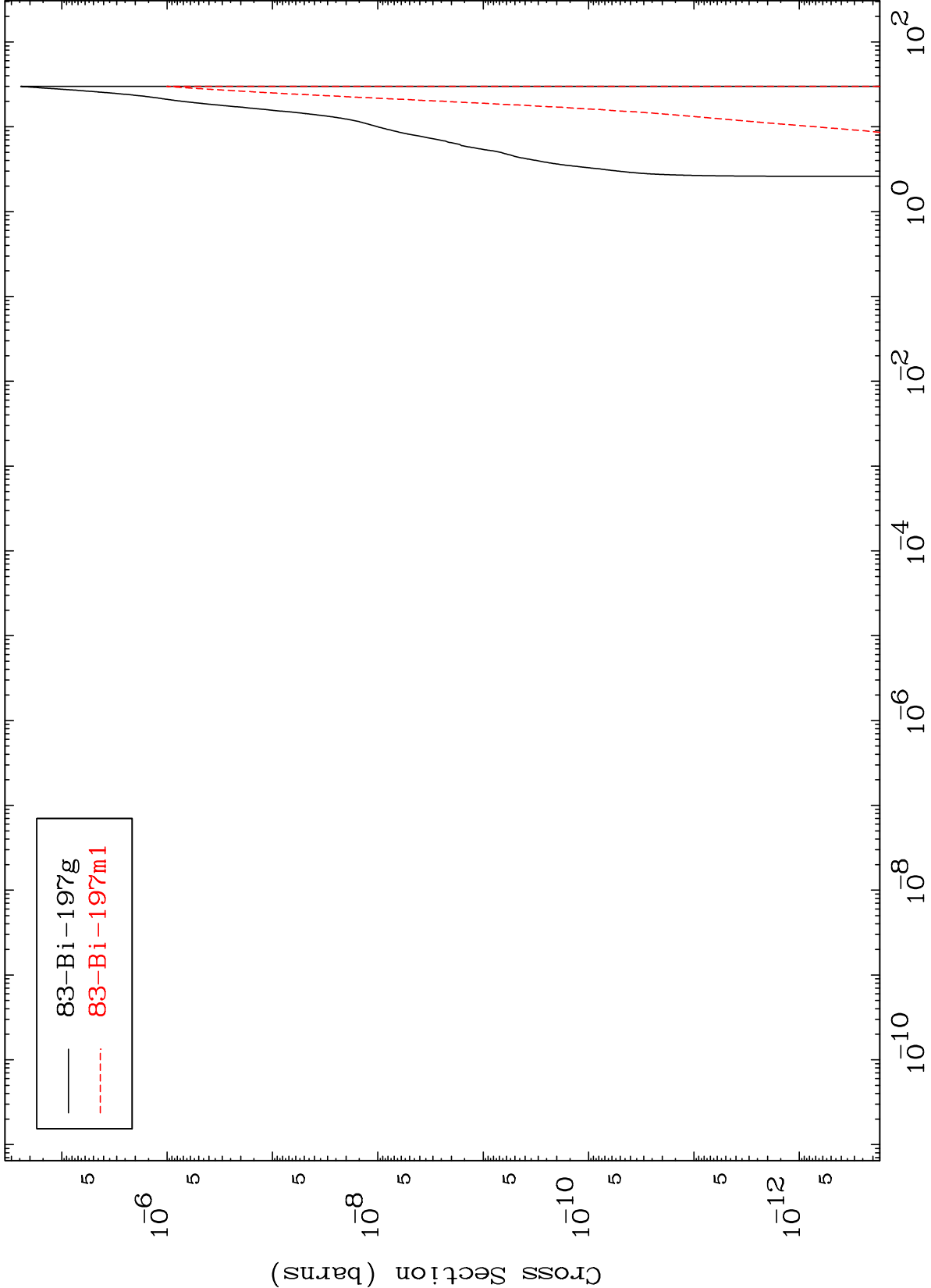
Incident Energy (MeV)

87-Fr-205

MAT 8704

87-Fr-205

(n,n') 2α
Radionuclide Production Cross Section



63

Incident Energy (MeV)

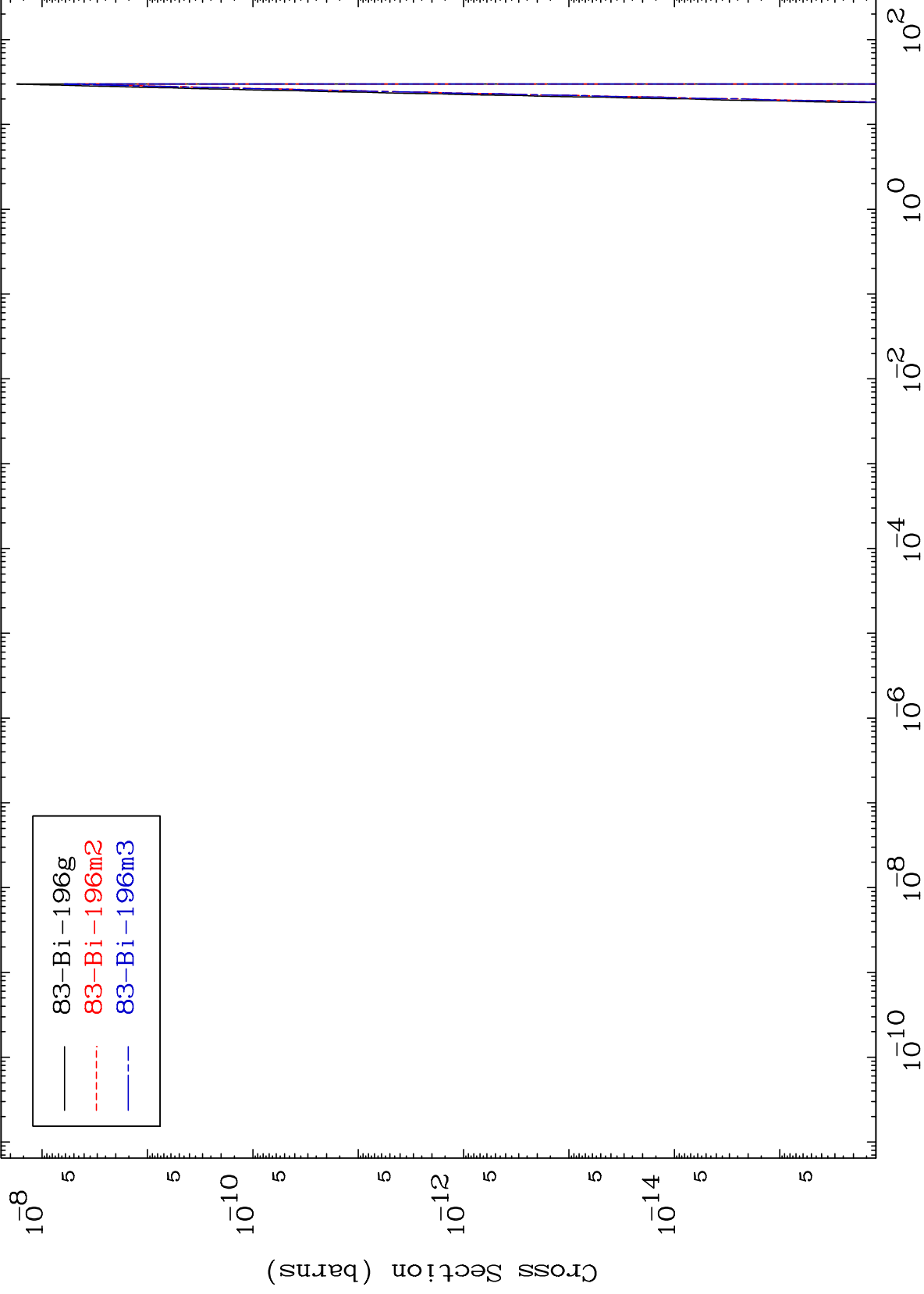
87-Fr-205

MAT 8704

(n,2n) 2 α

87-Fr-205

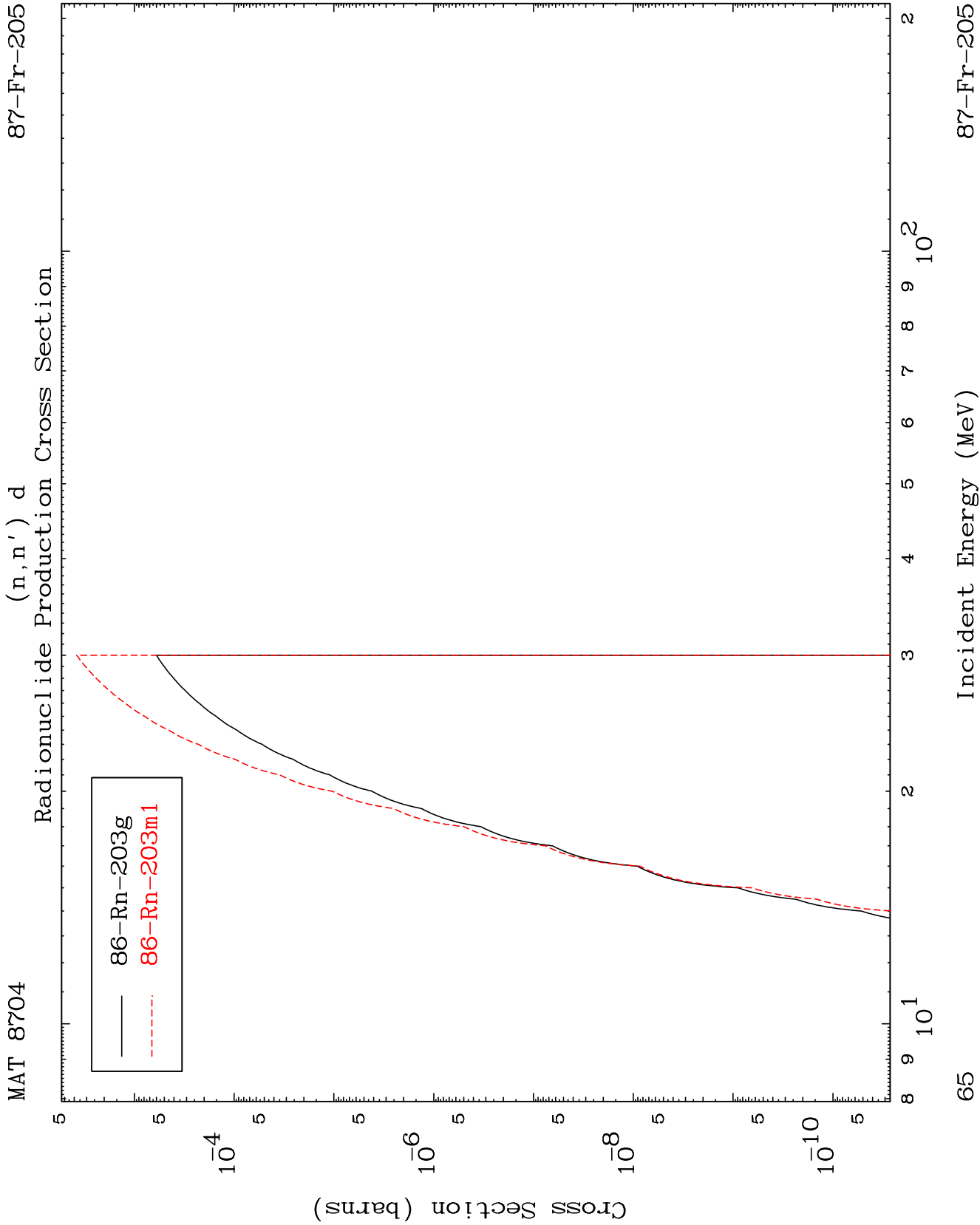
Radionuclide Production Cross Section



64

Incident Energy (MeV)

87-Fr-205

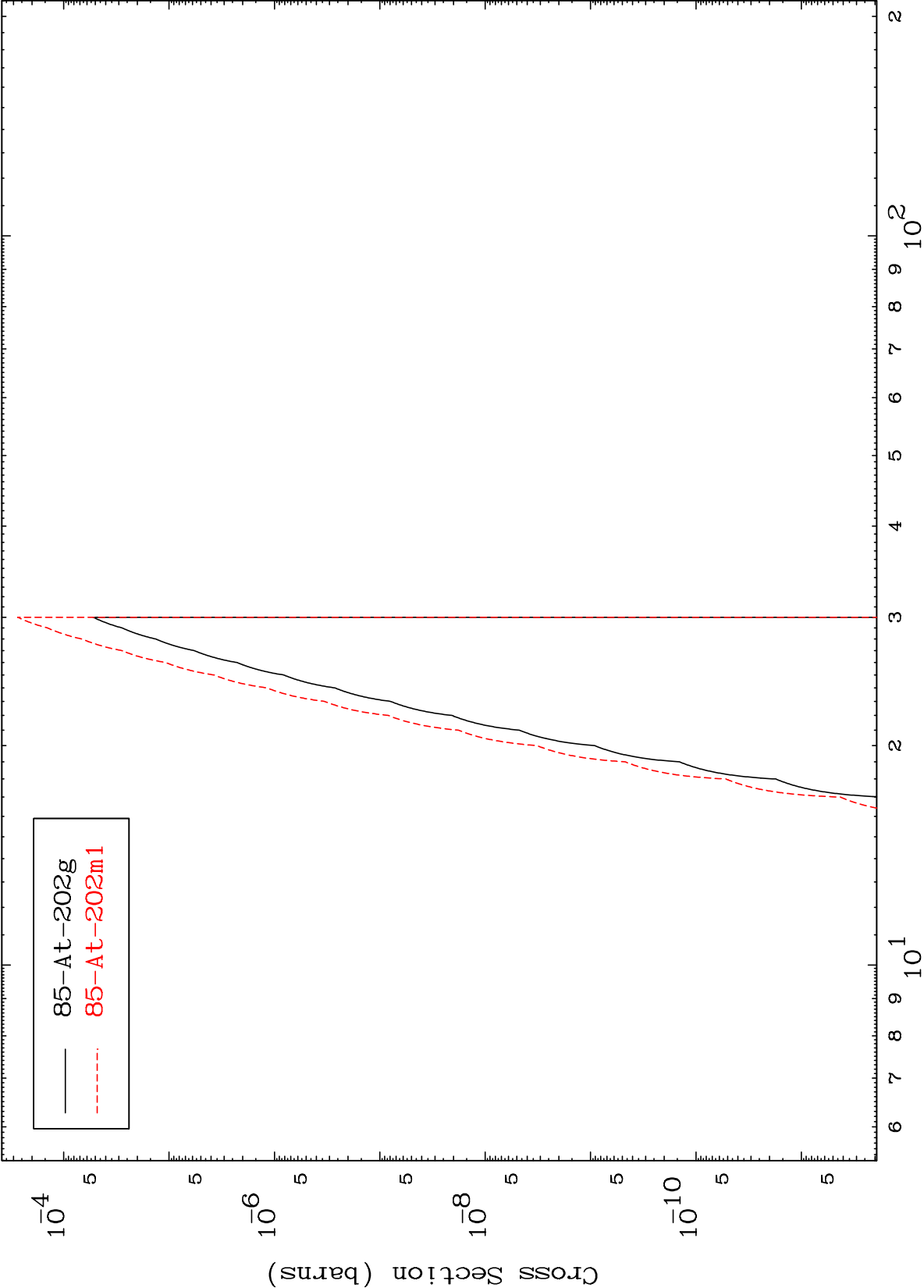


MAT 8704

(n,n') He-3

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Radionuclide Production Cross Section



66

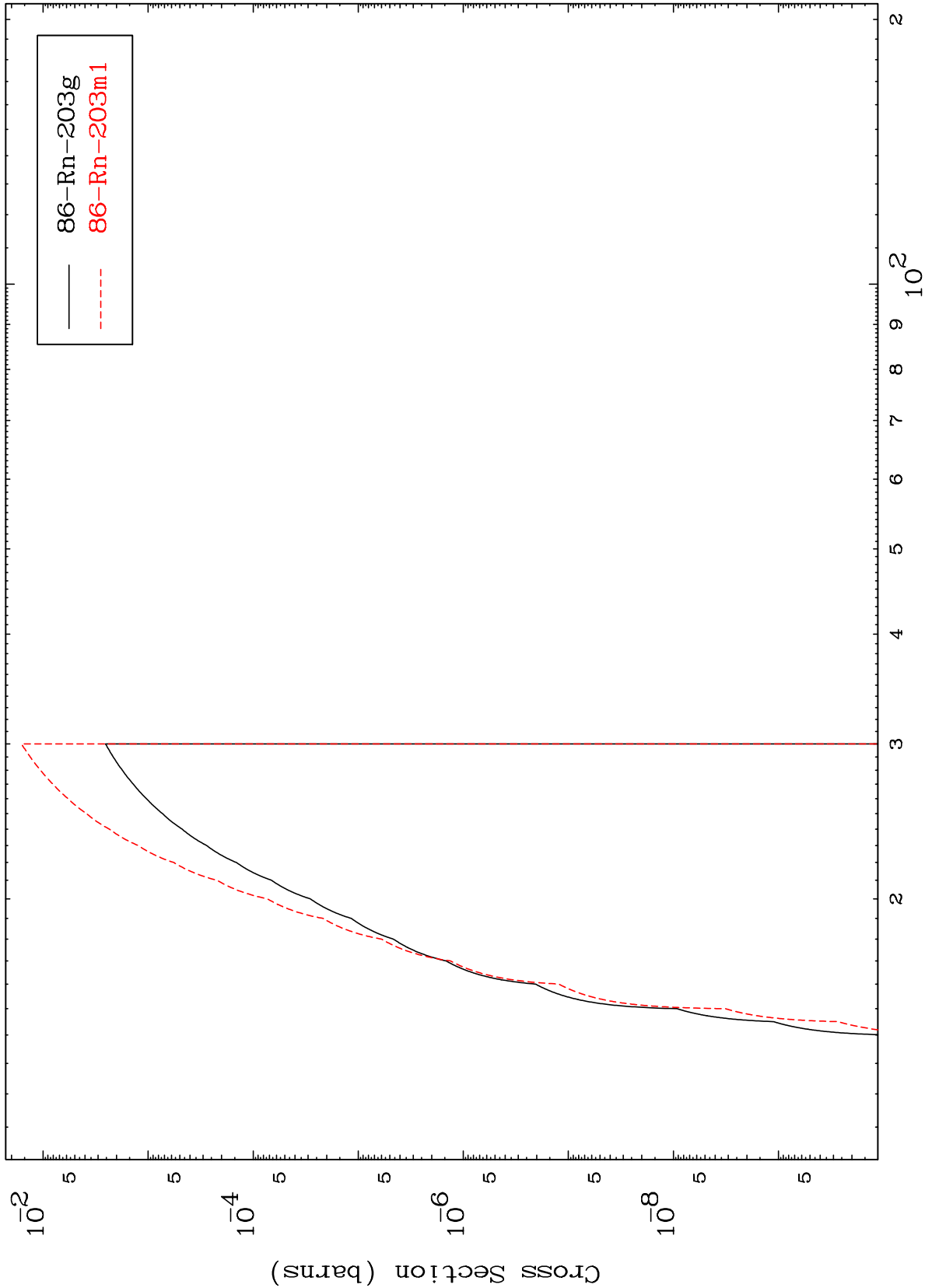
Incident Energy (MeV)

87-Fr-205

MAT 8704

87-Fr-205

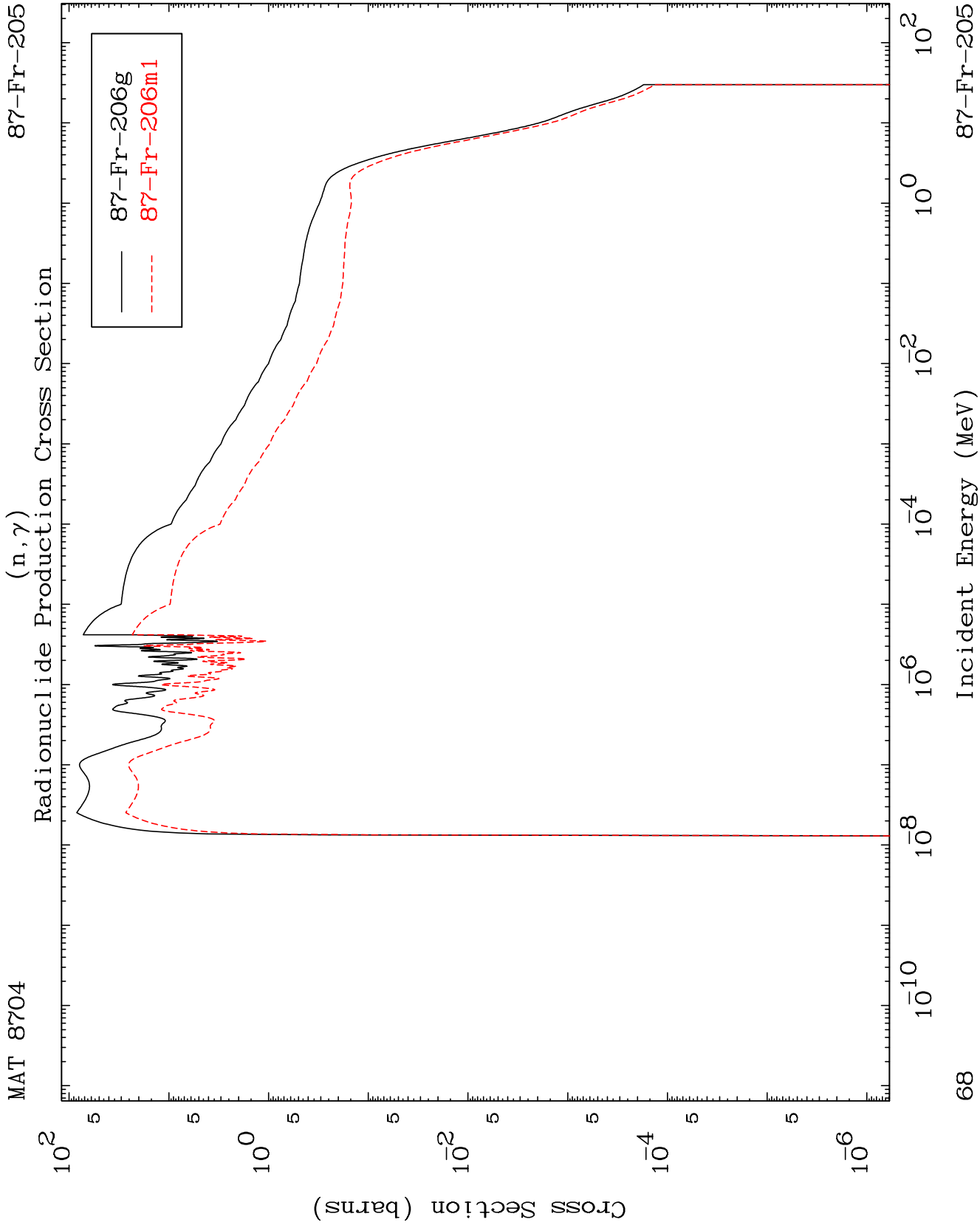
(n,2n) p
Radionuclide Production Cross Section

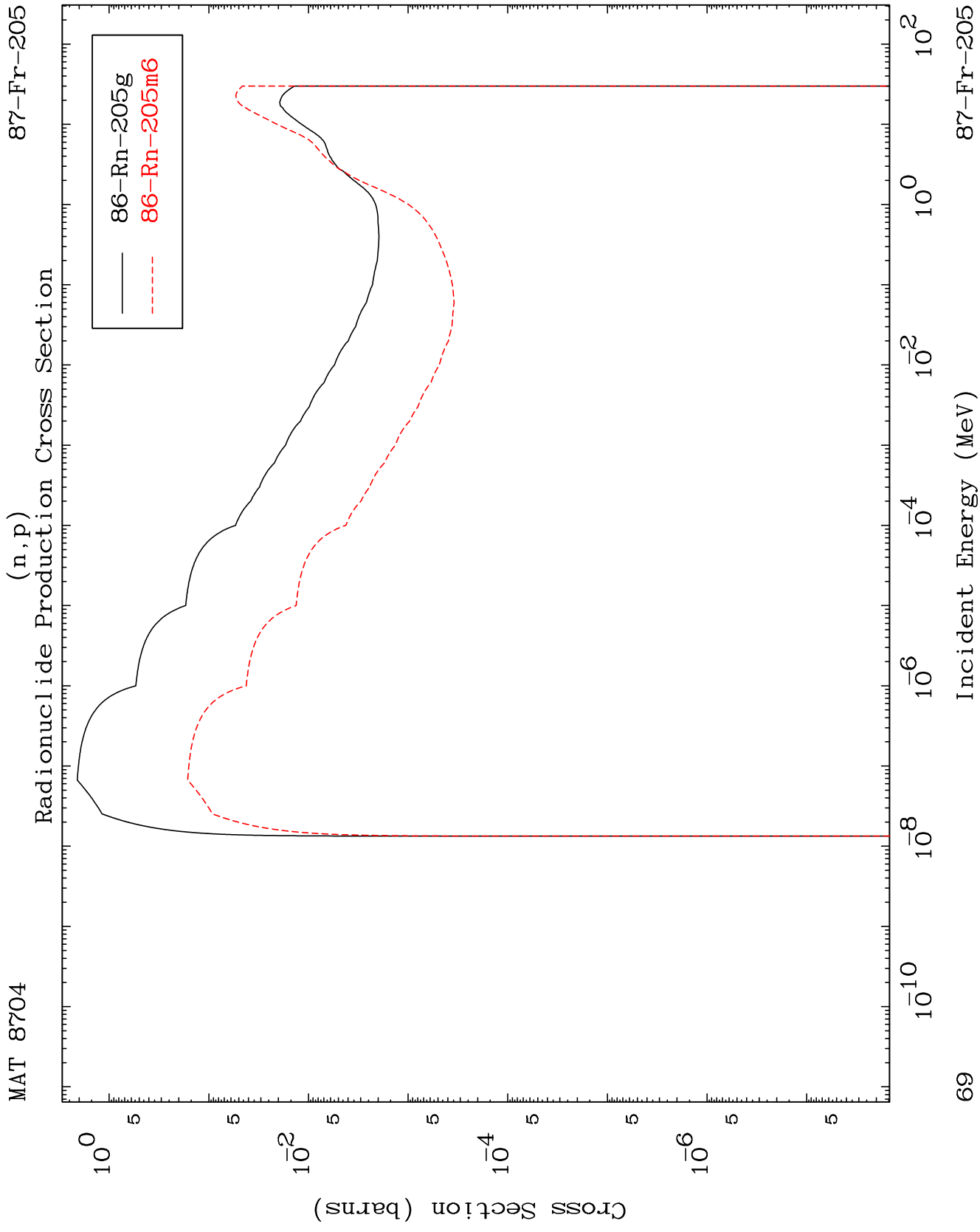


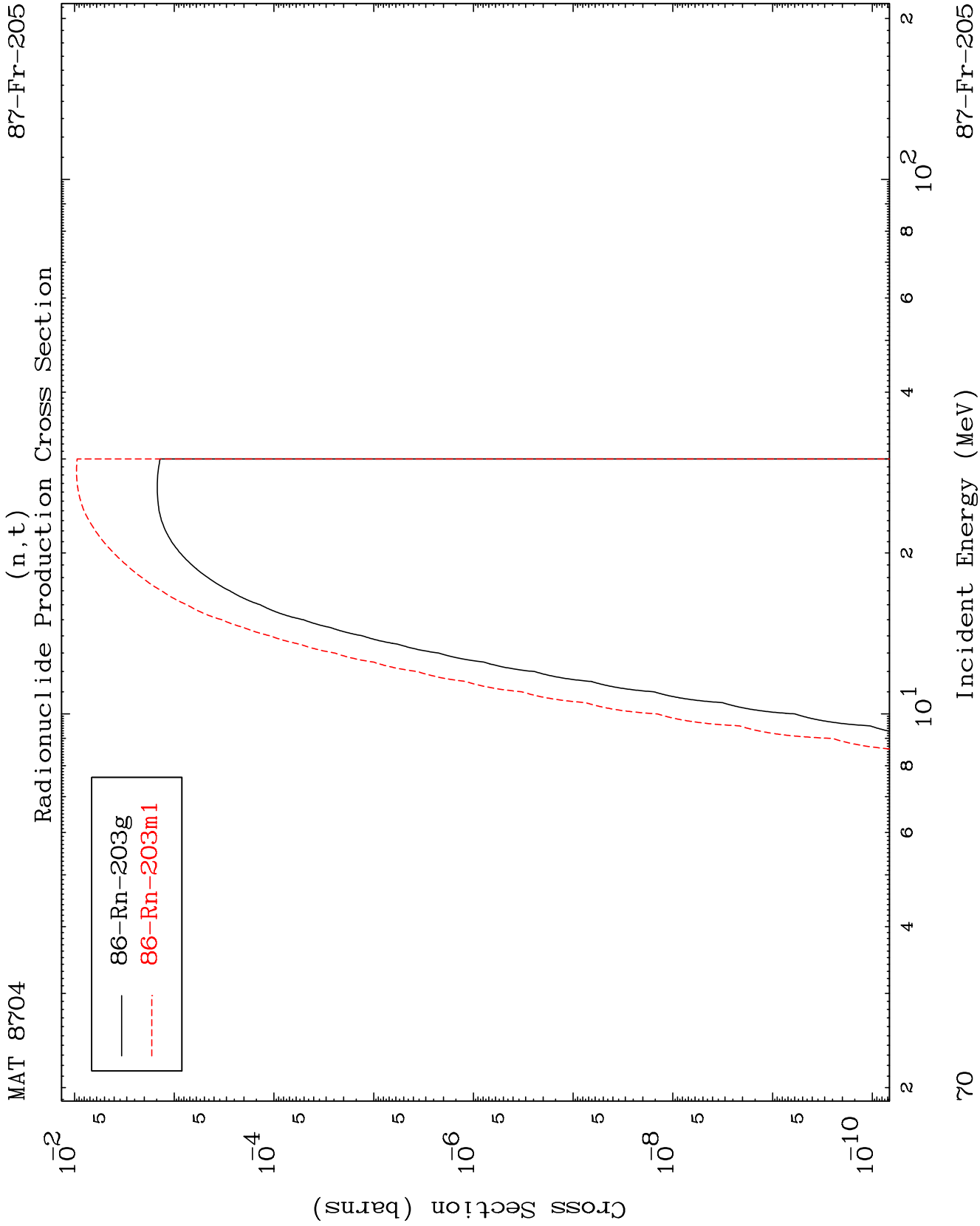
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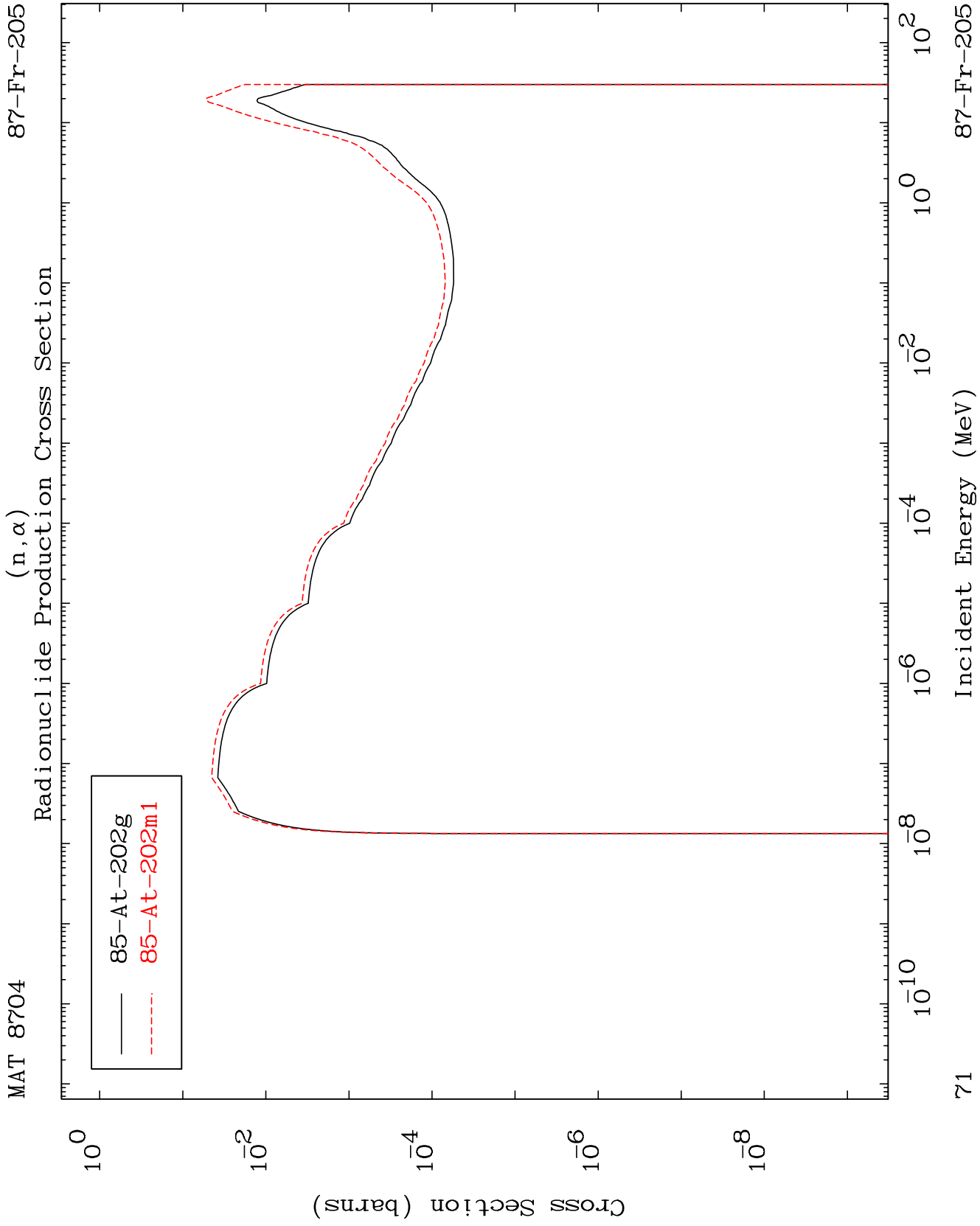
Incident Energy (MeV)

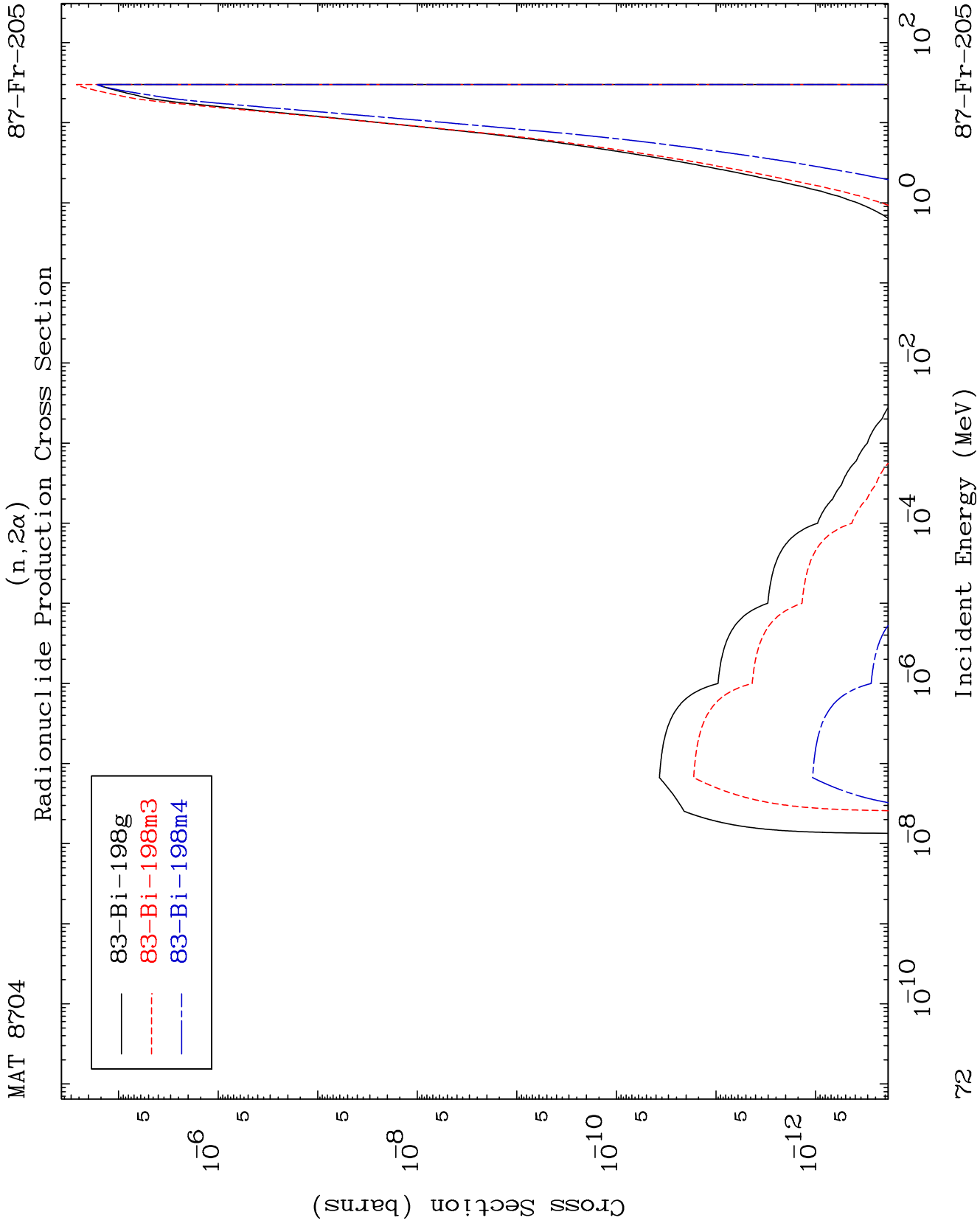
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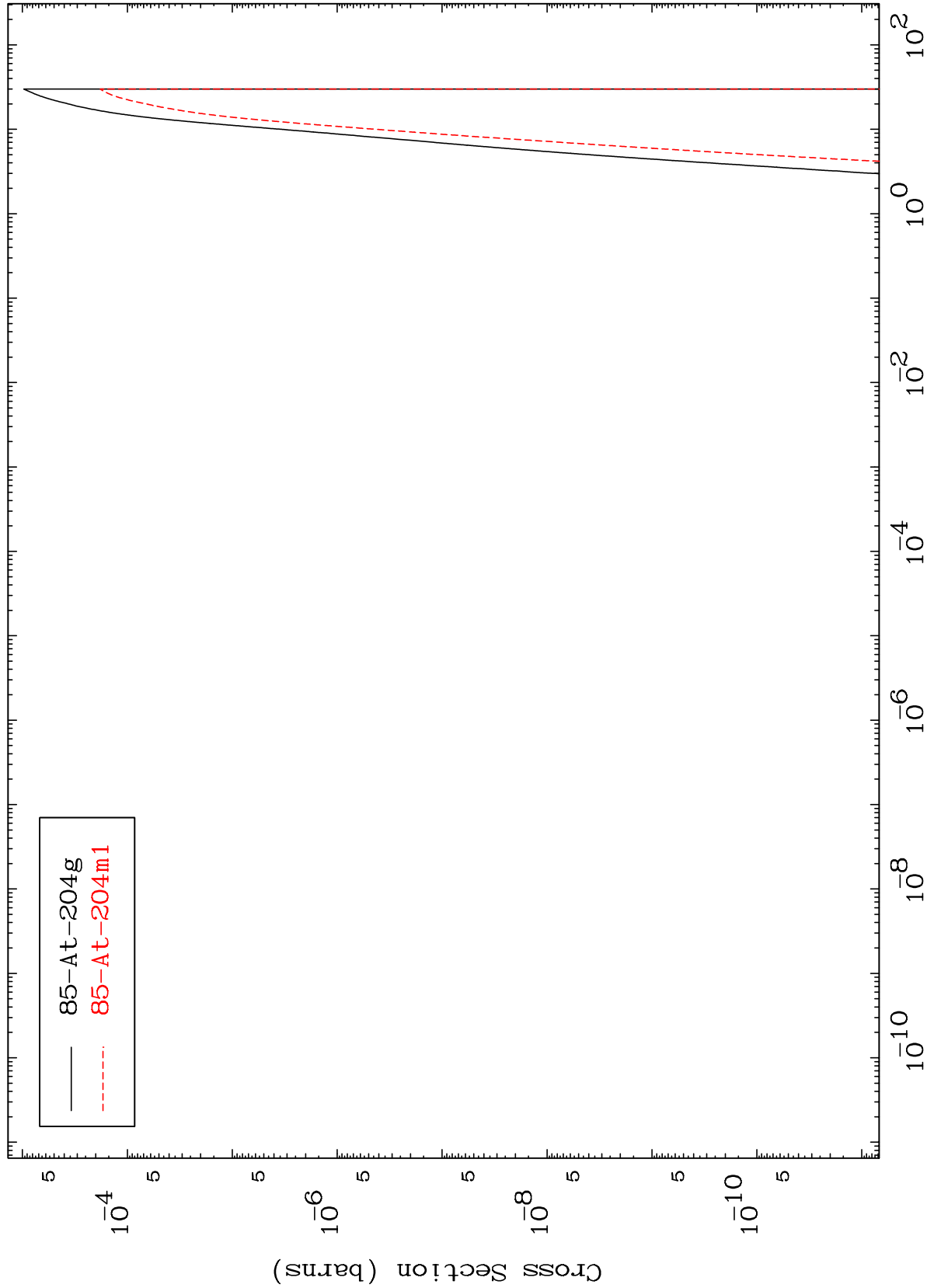




MAT 8704

87-Fr-205

(n,2p)
Radionuclide Production Cross Section



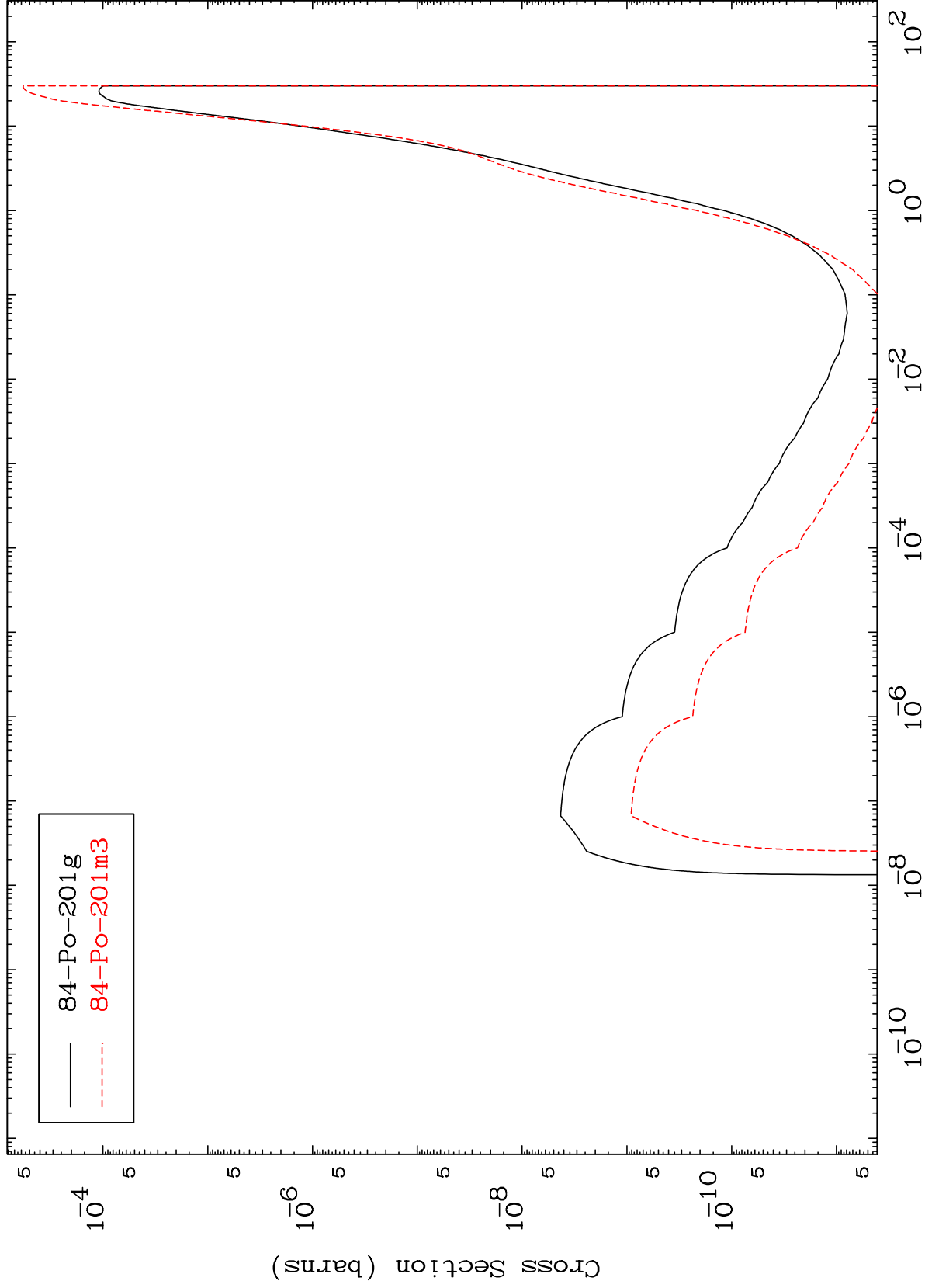
73

87-Fr-205

MAT 8704

87-Fr-205

(n,p) α
Radionuclide Production Cross Section



74

87-Fr-205

