

Program EVALPLOT
(Version 2021-1)

by

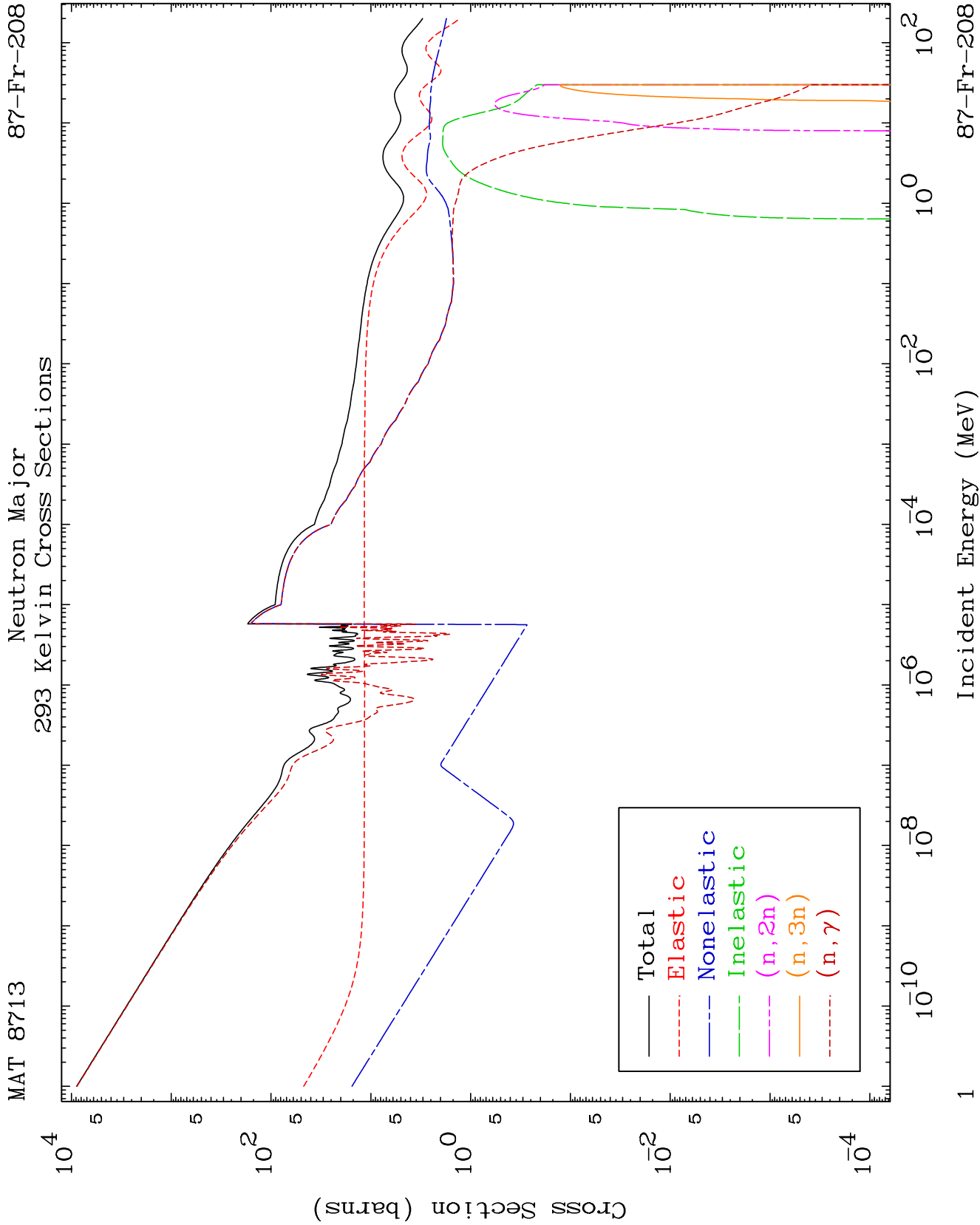
Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail: redcullen1@comcast.net
Web: redcullen1.net/HOMEPAGE.NEW

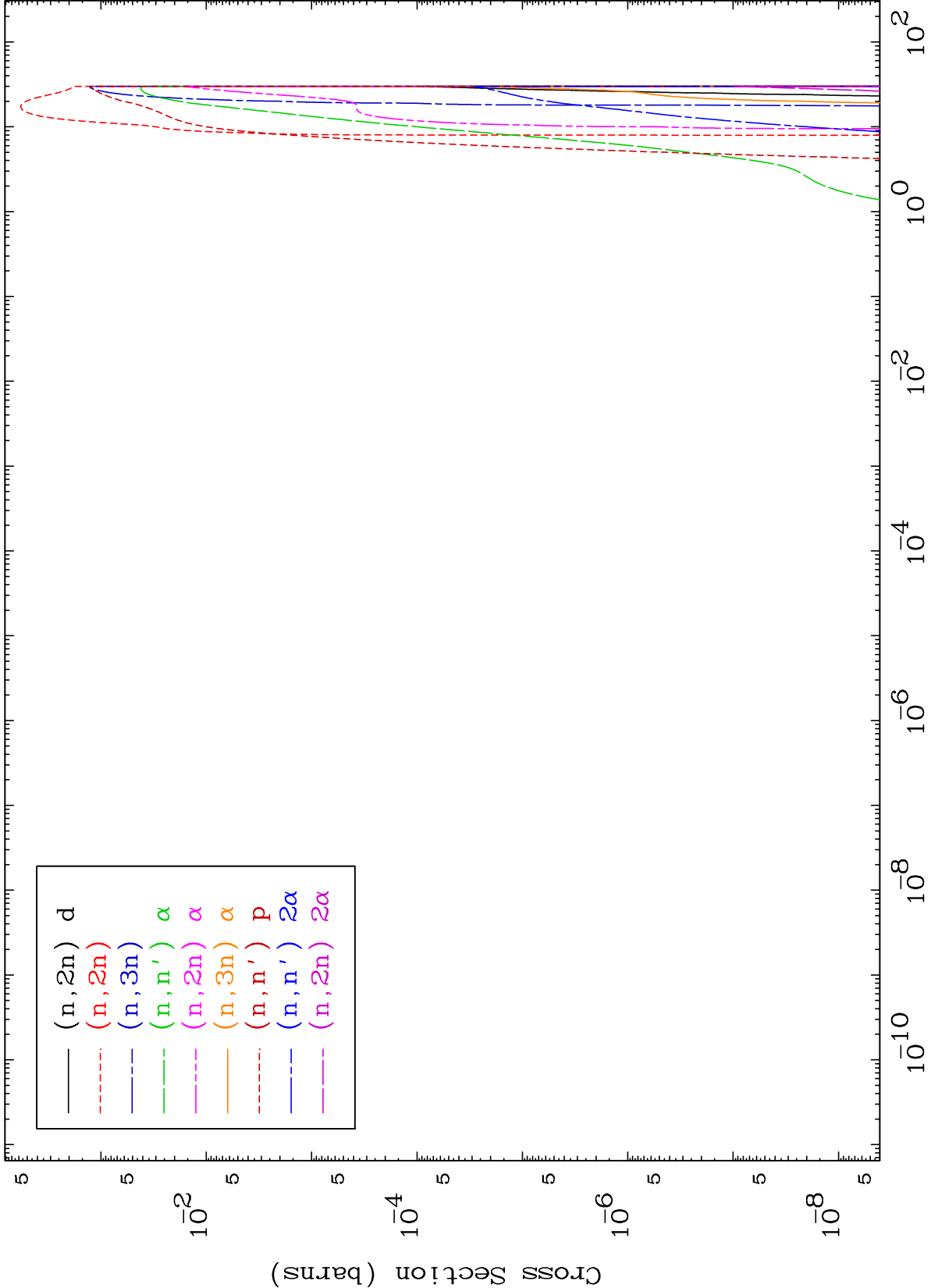
Press Mouse Button to Start

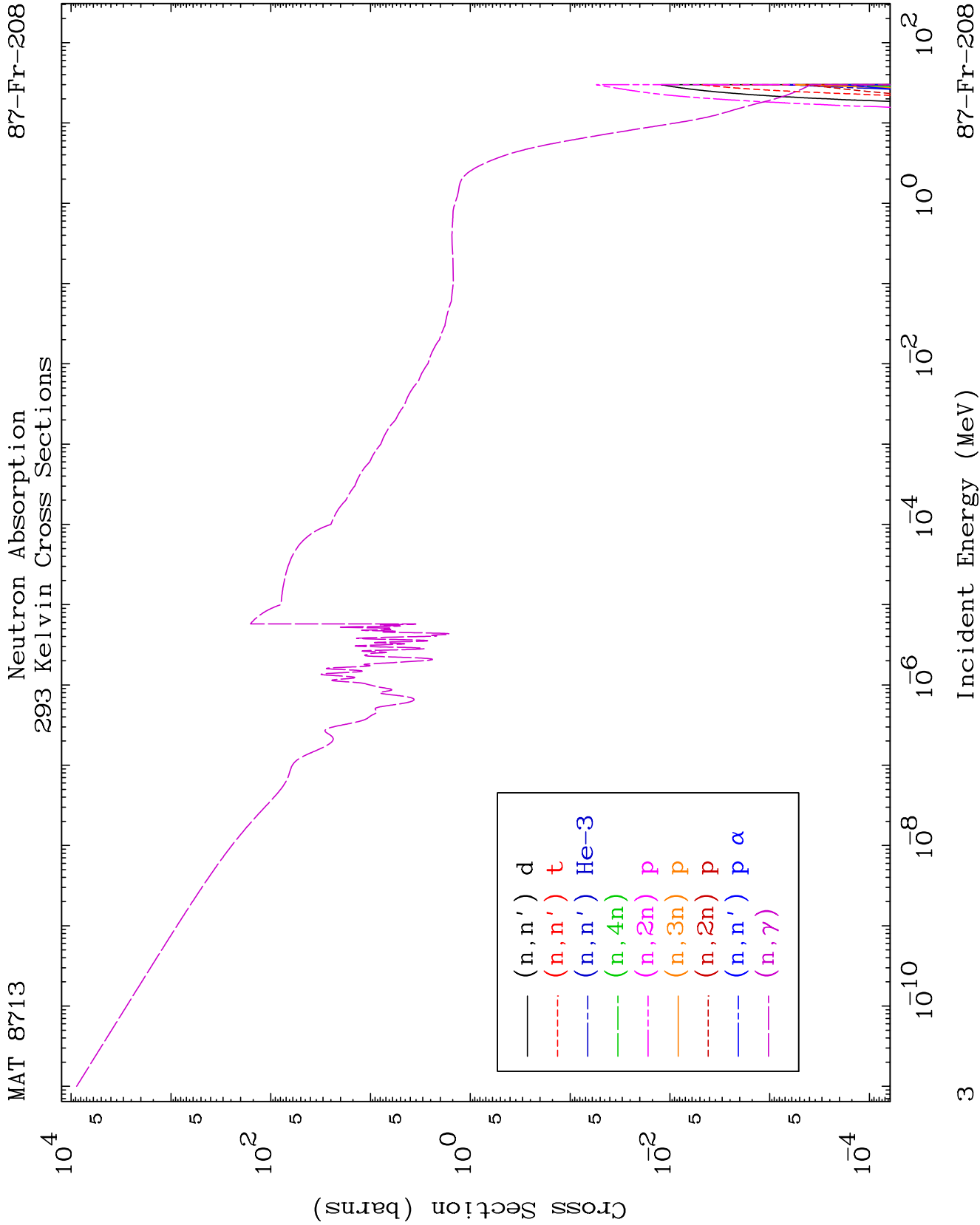


MAT 8713

Neutron Absorption
293 Kelvin Cross Sections

87-Fr-208

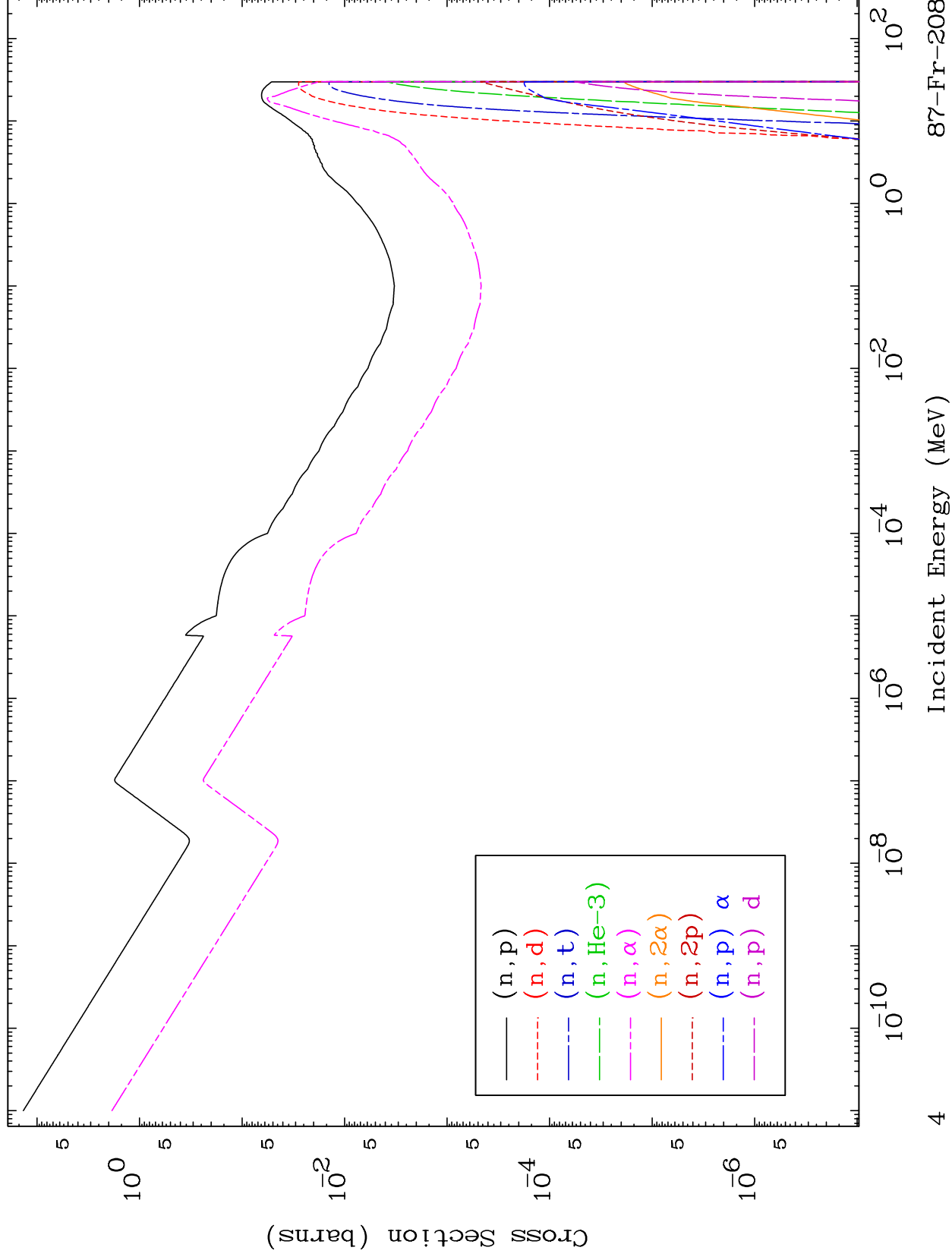


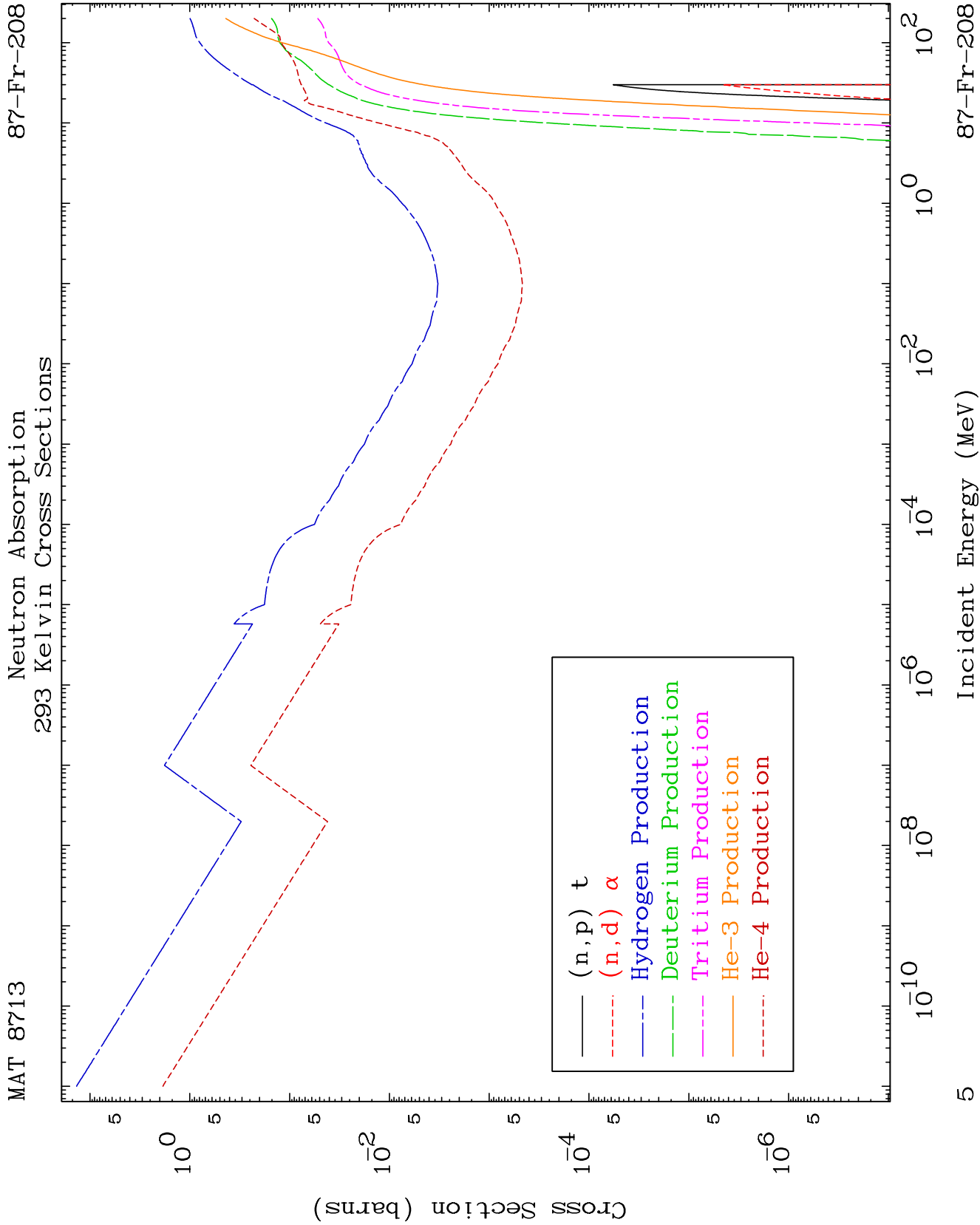


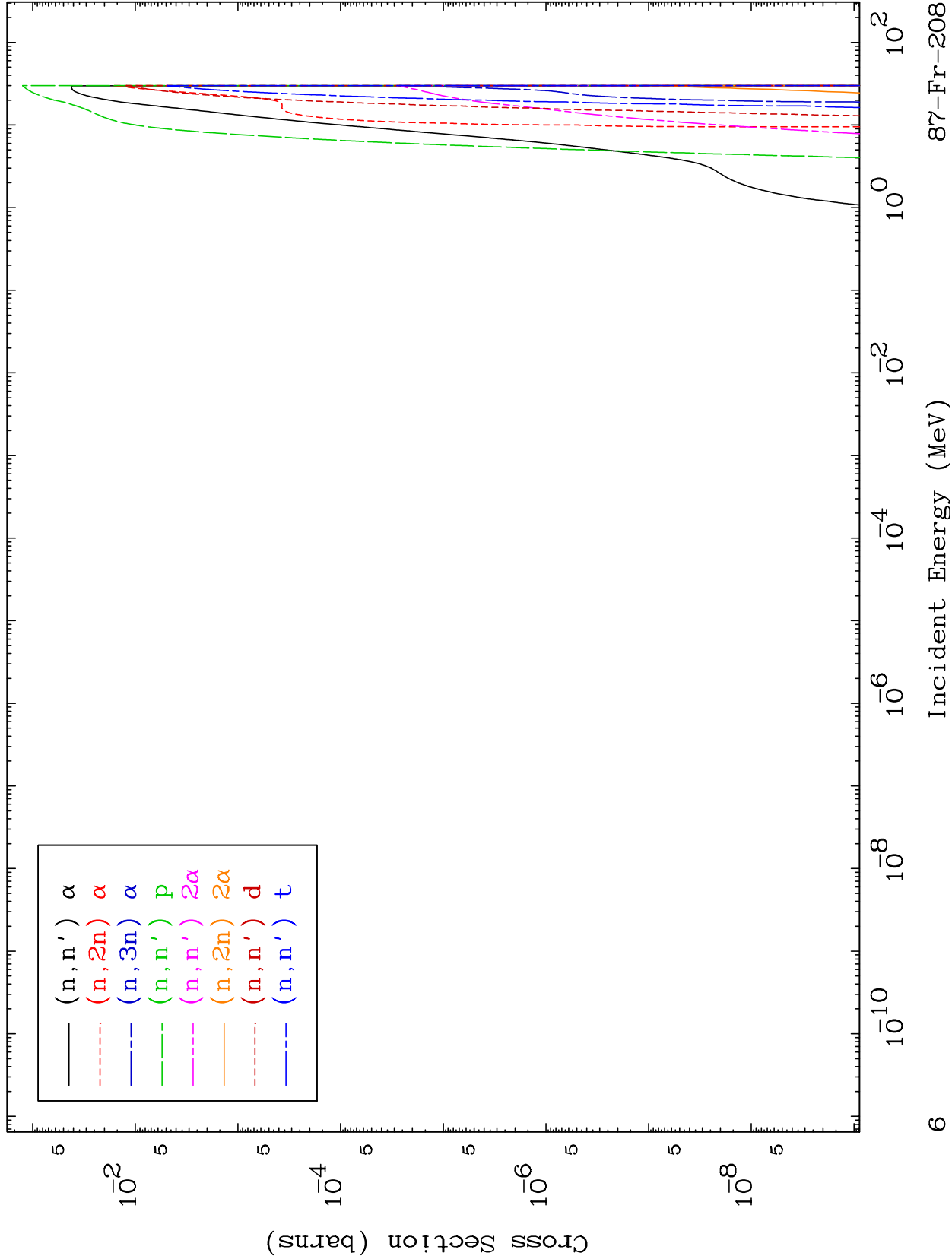
MAT 8713

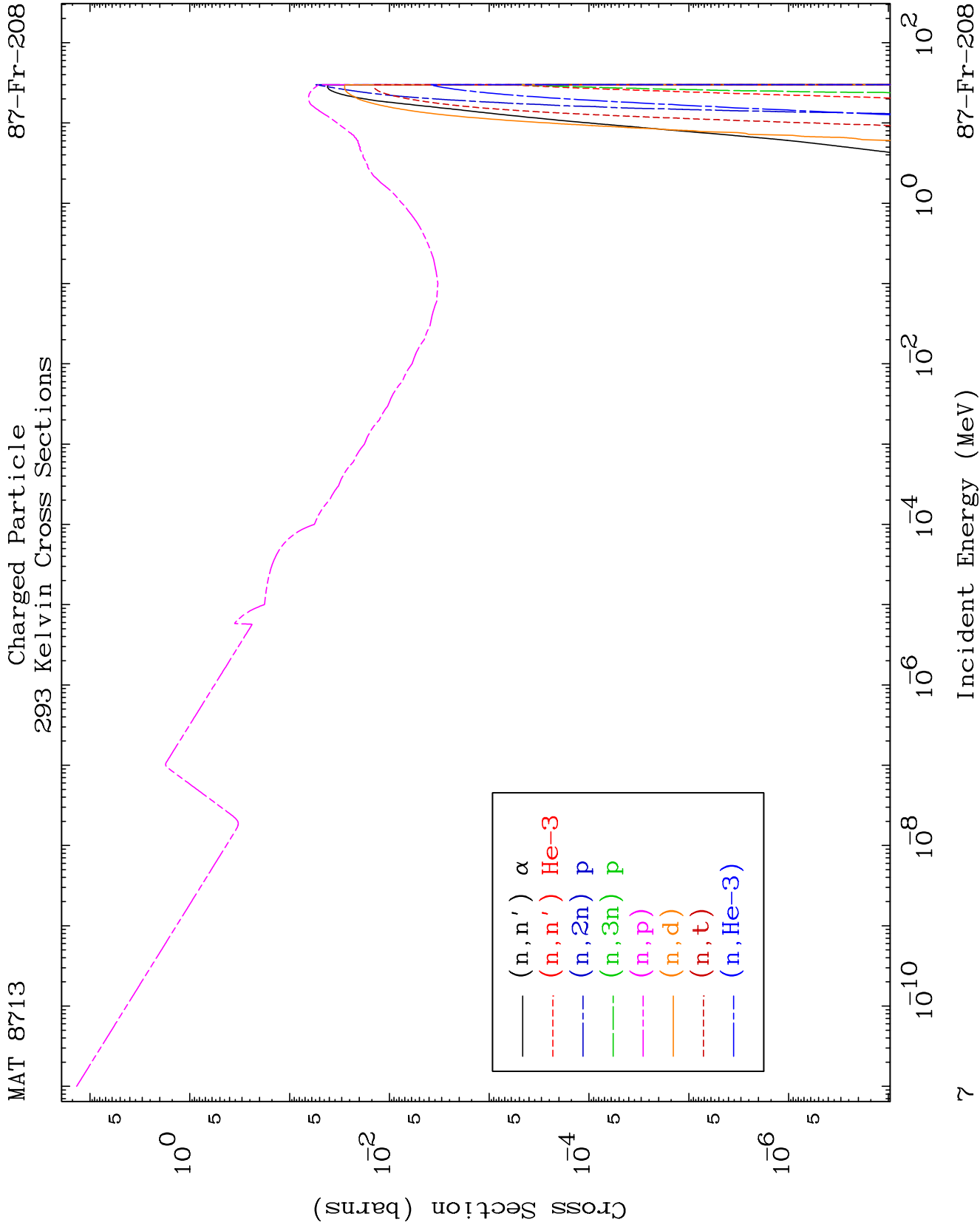
Neutron Absorption
293 Kelvin Cross Sections

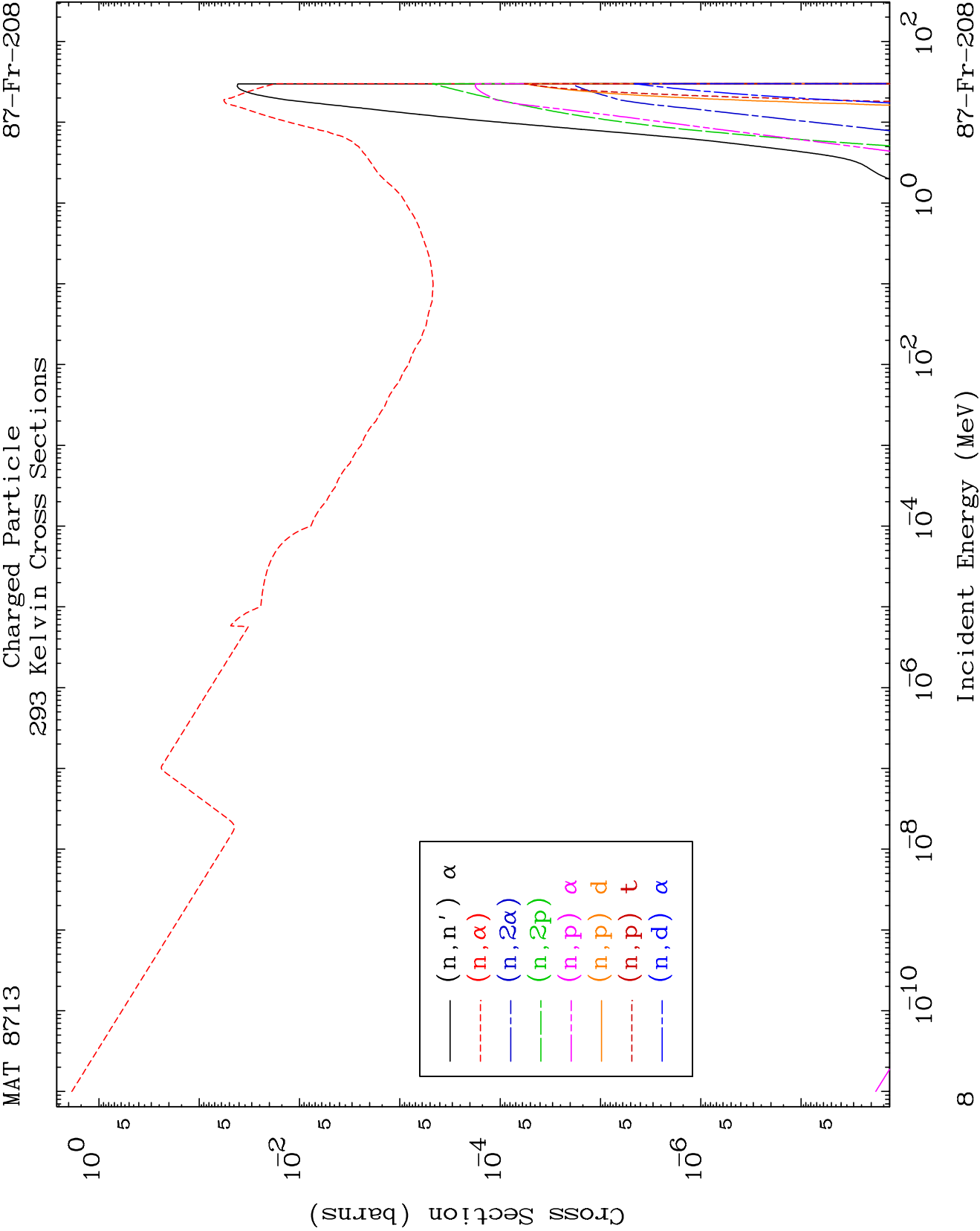
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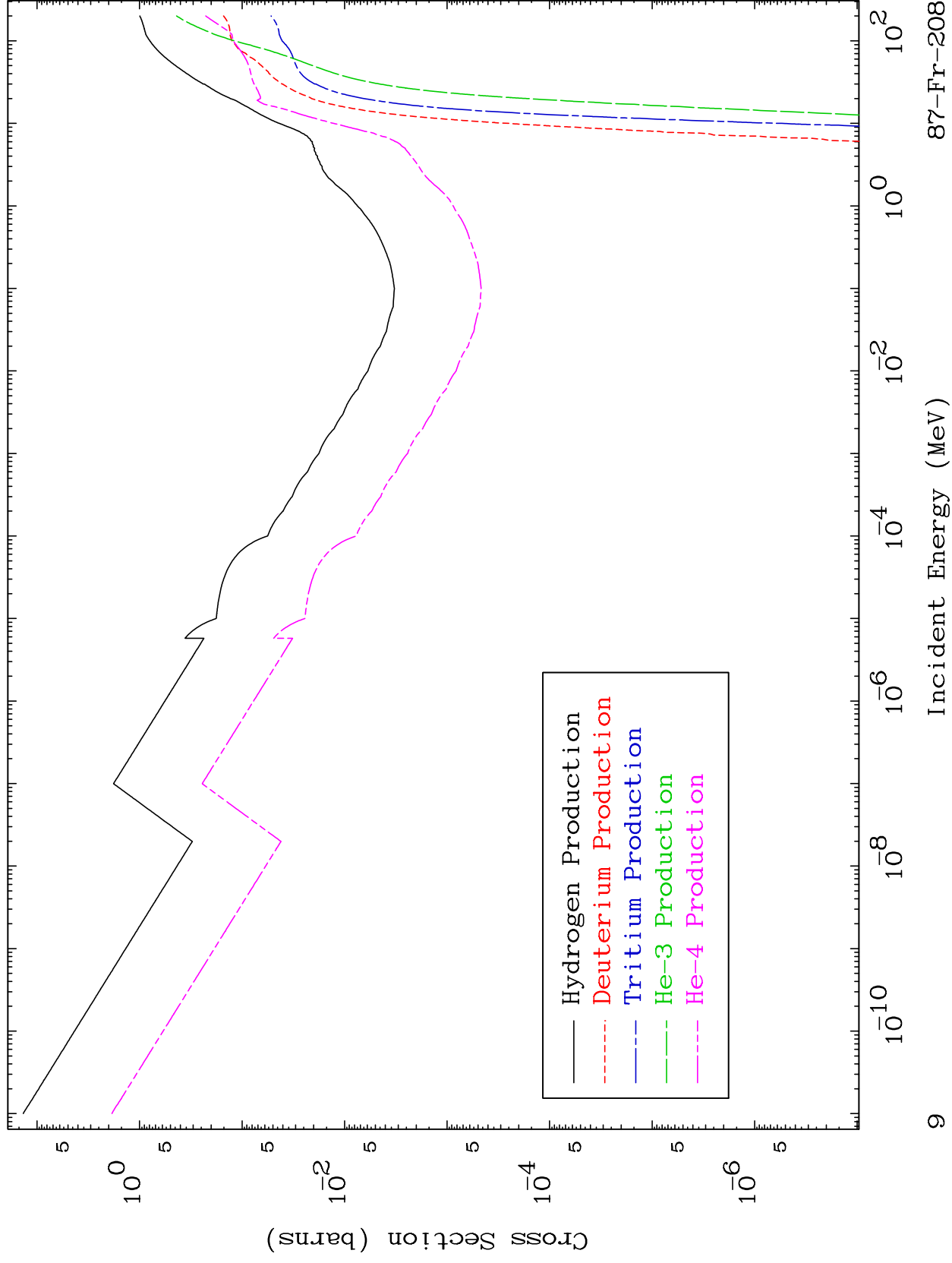


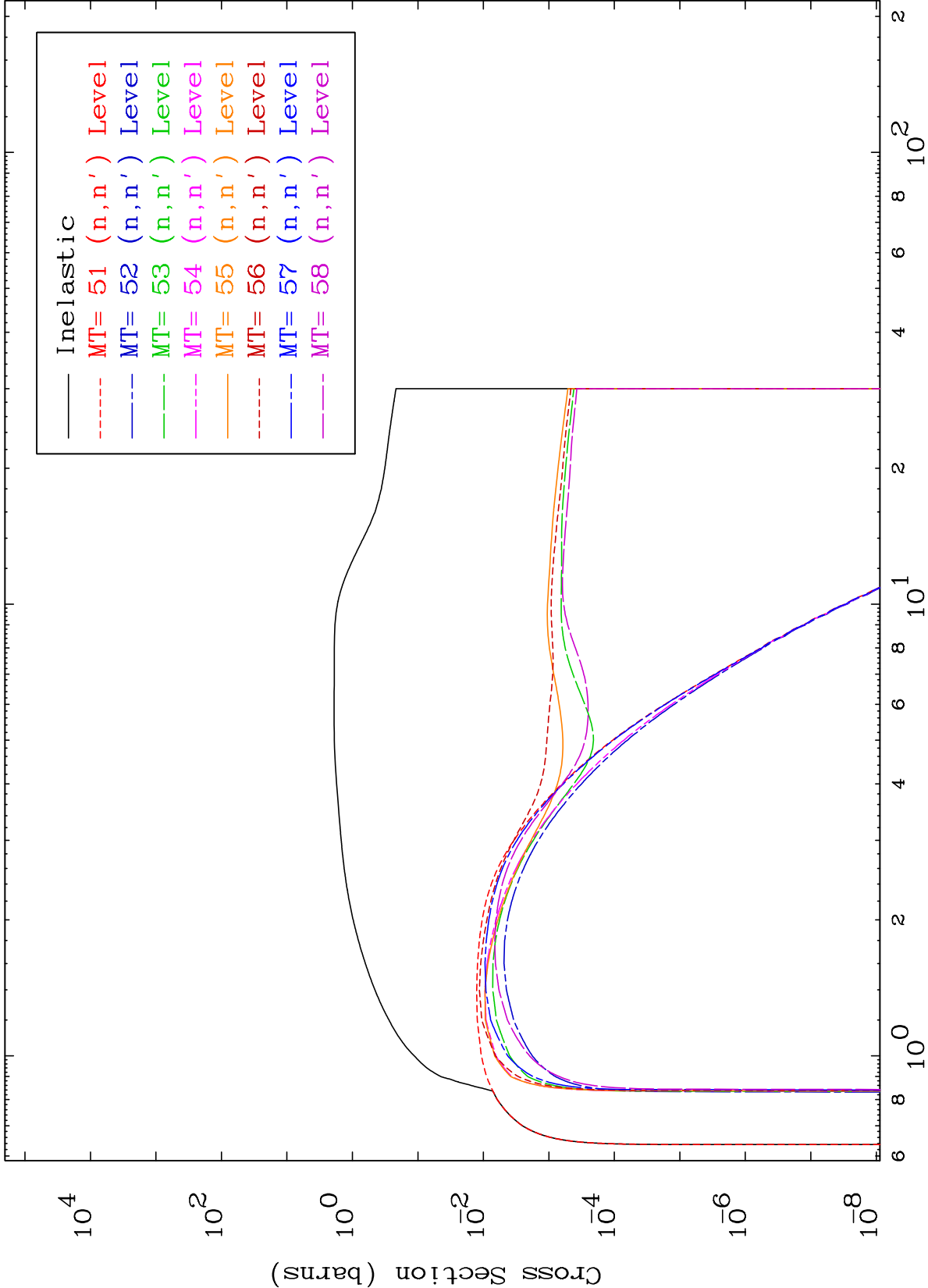


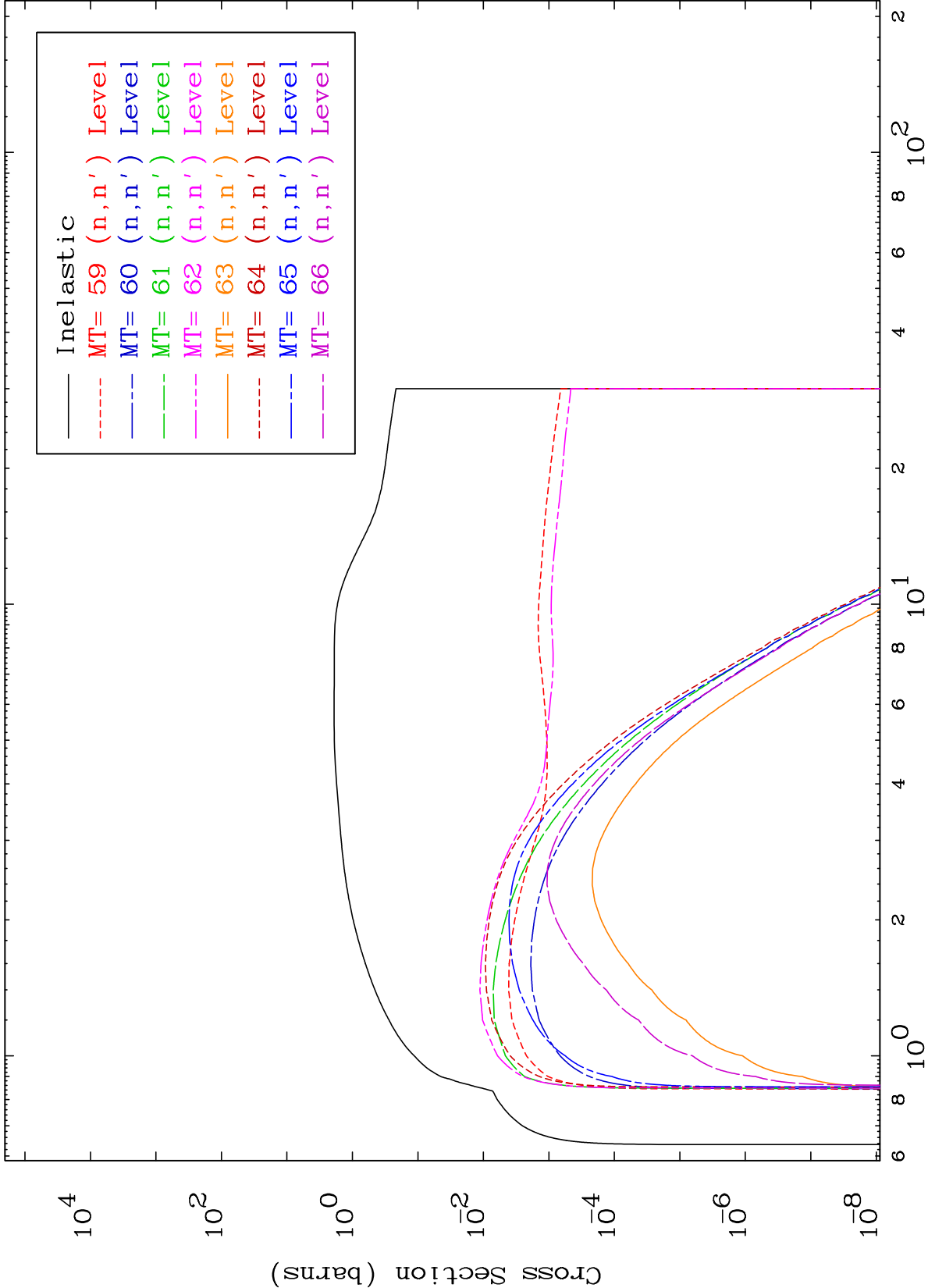


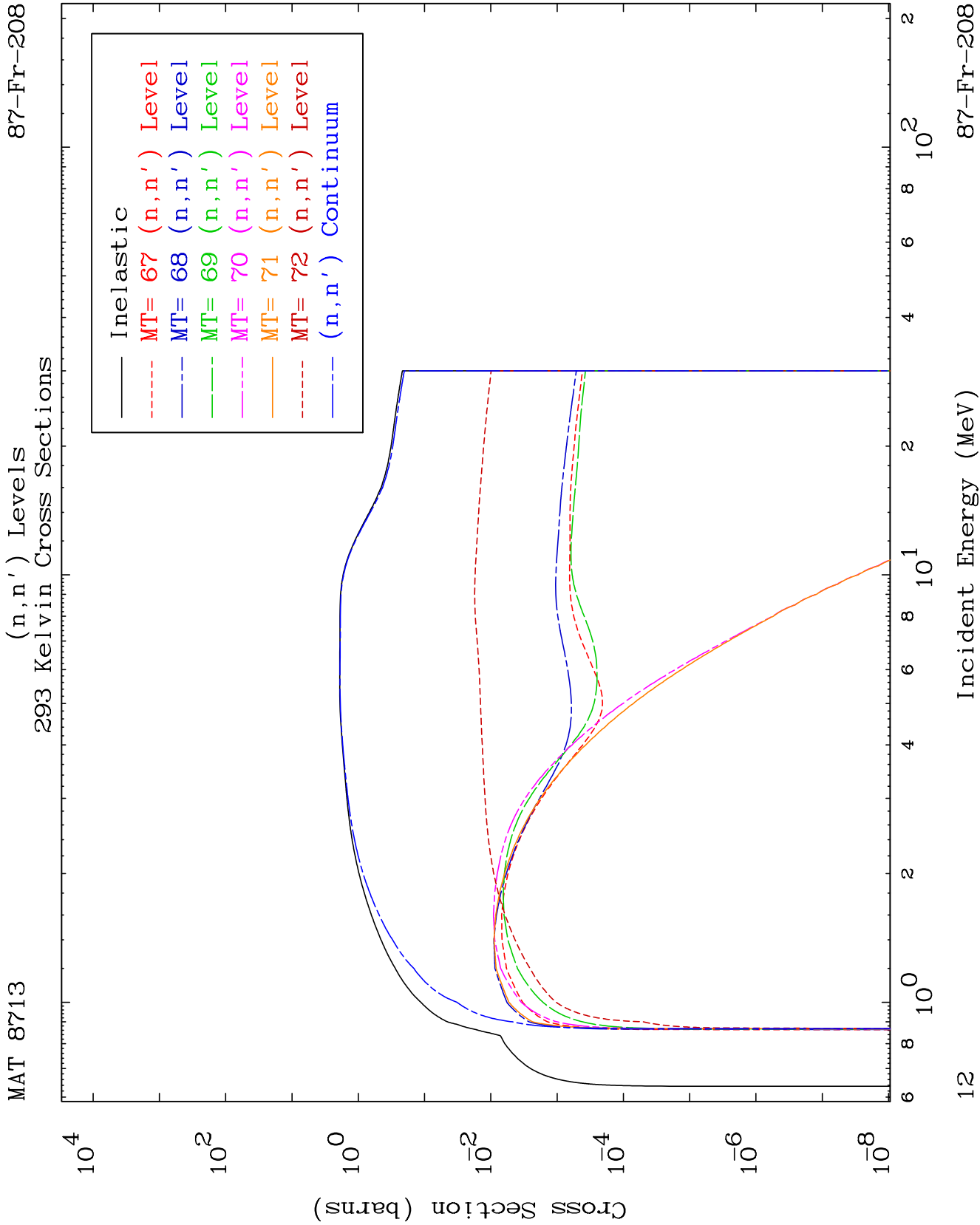








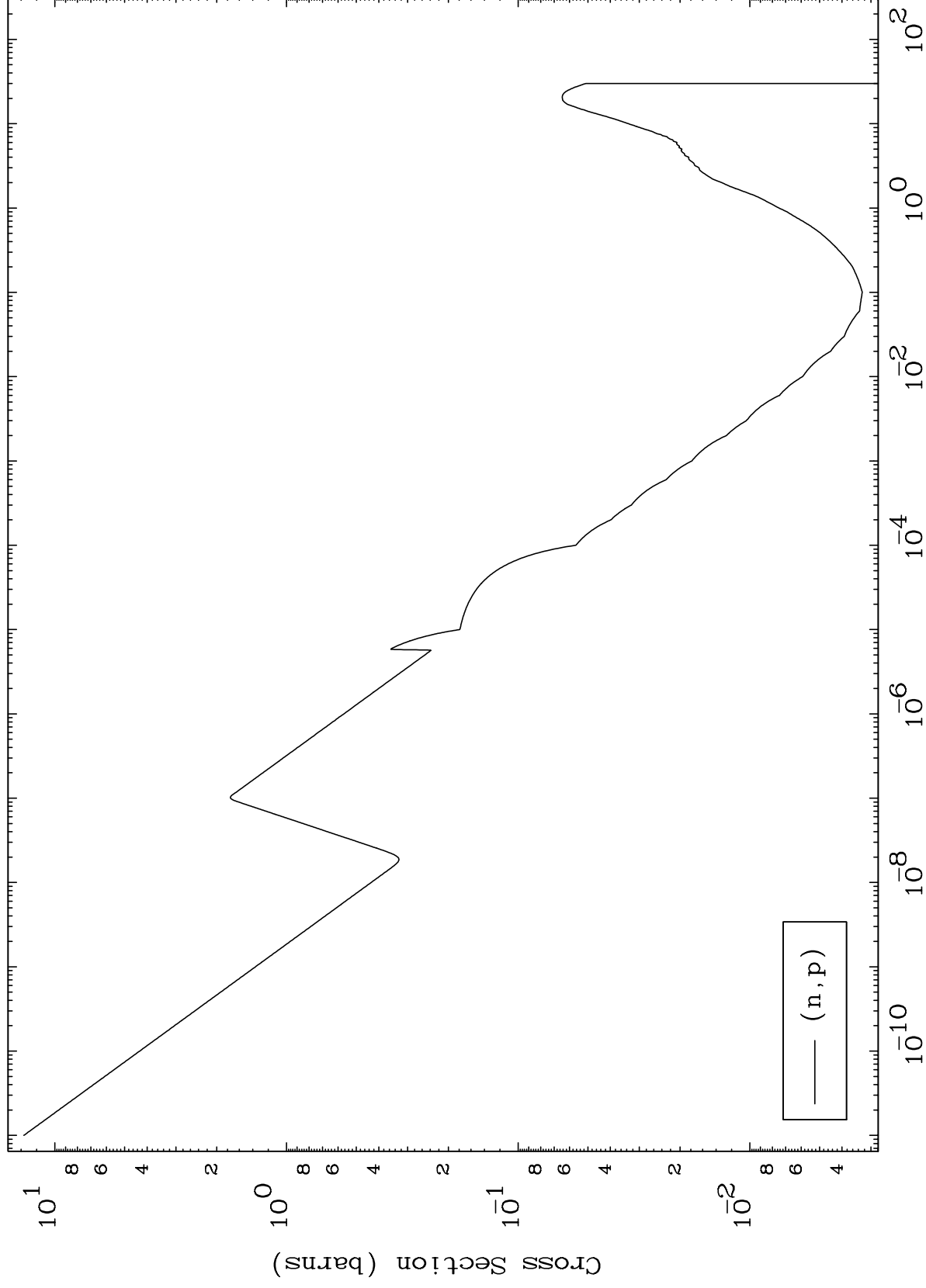




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(n,p) Levels
293 Kelvin Cross Sections



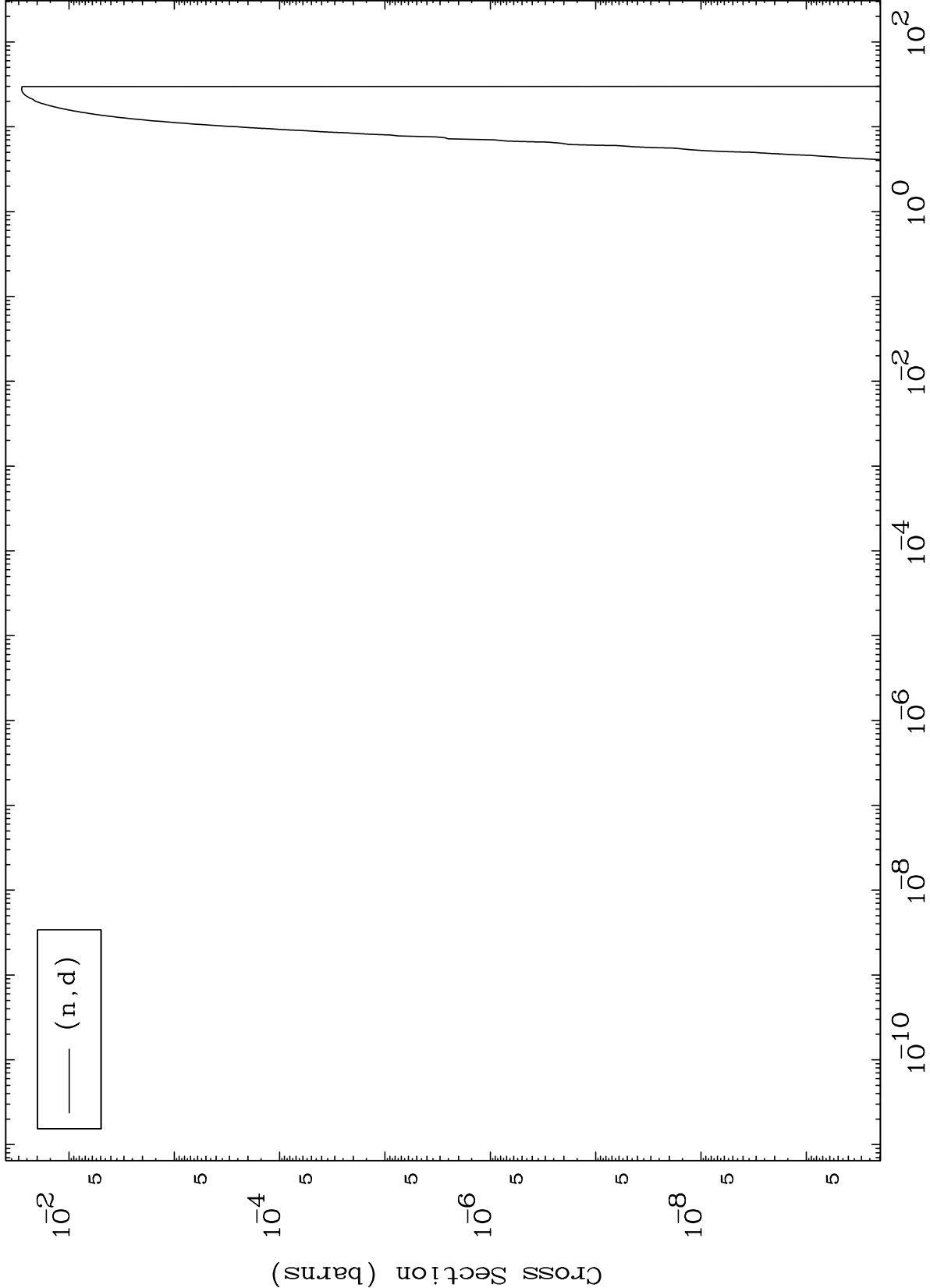
13

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(n,d) Levels
293 Kelvin Cross Sections

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14

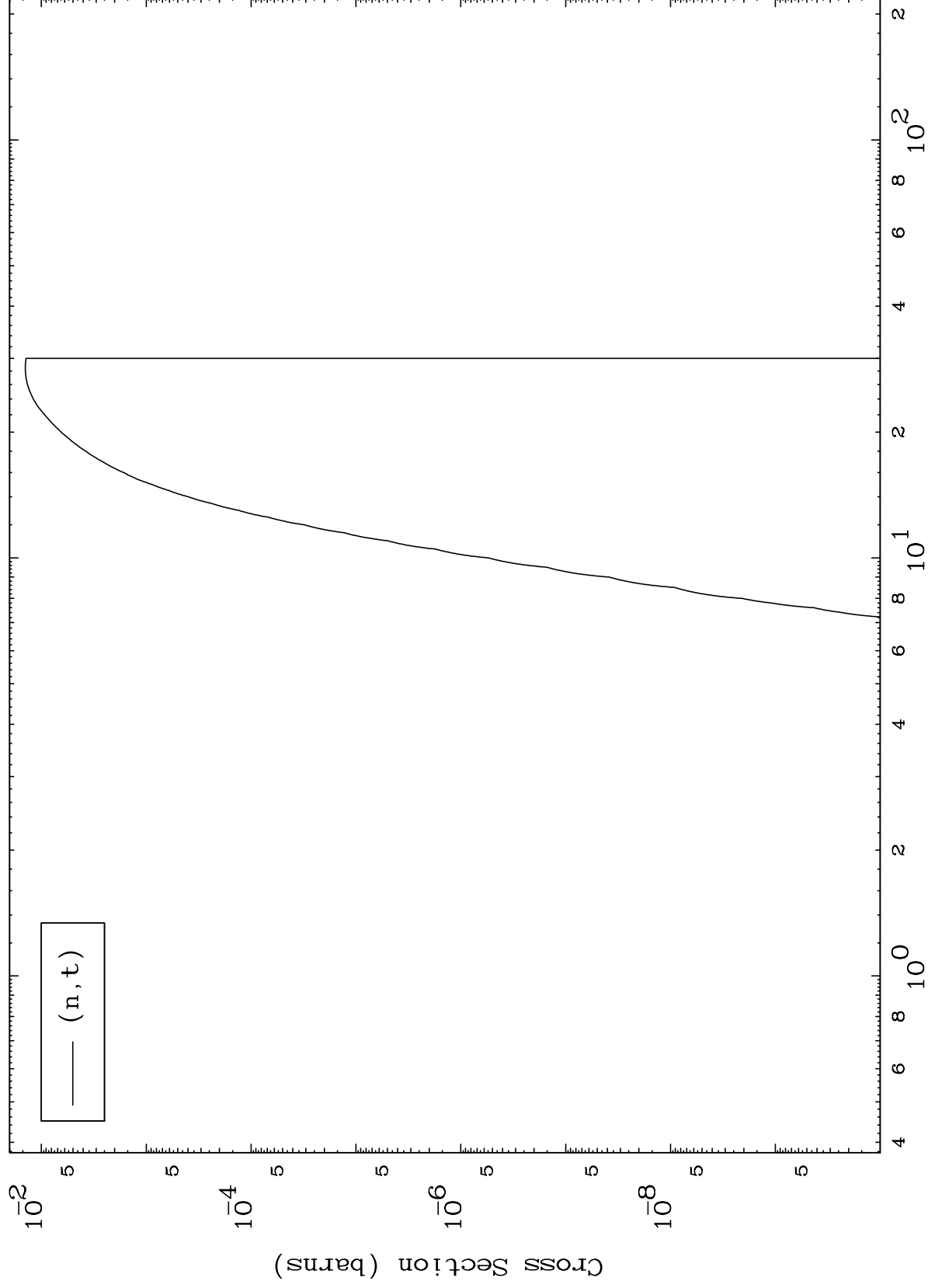
Incident Energy (MeV)

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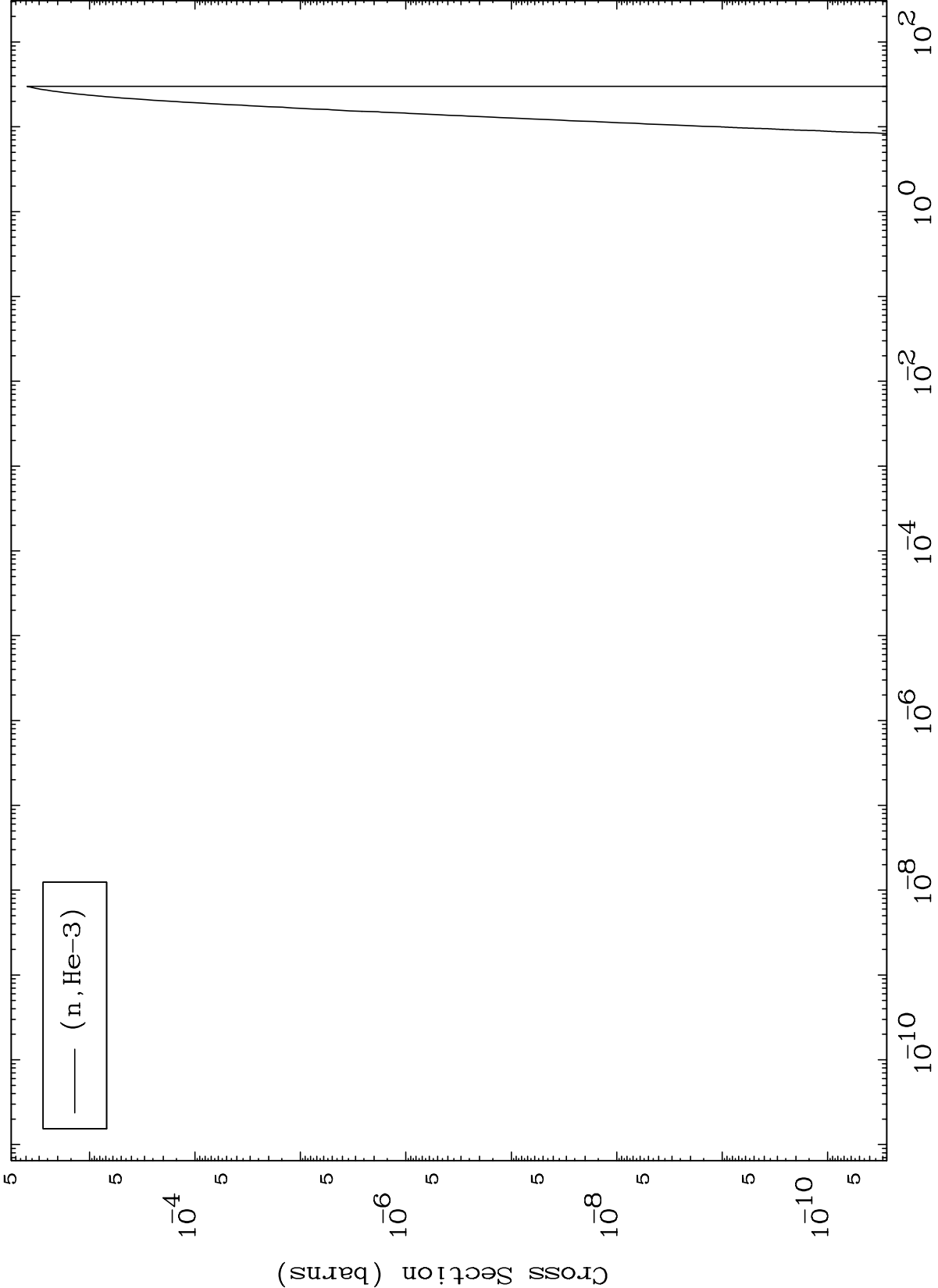
(n,t) Levels
293 Kelvin Cross Sections



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Incident Energy (MeV)

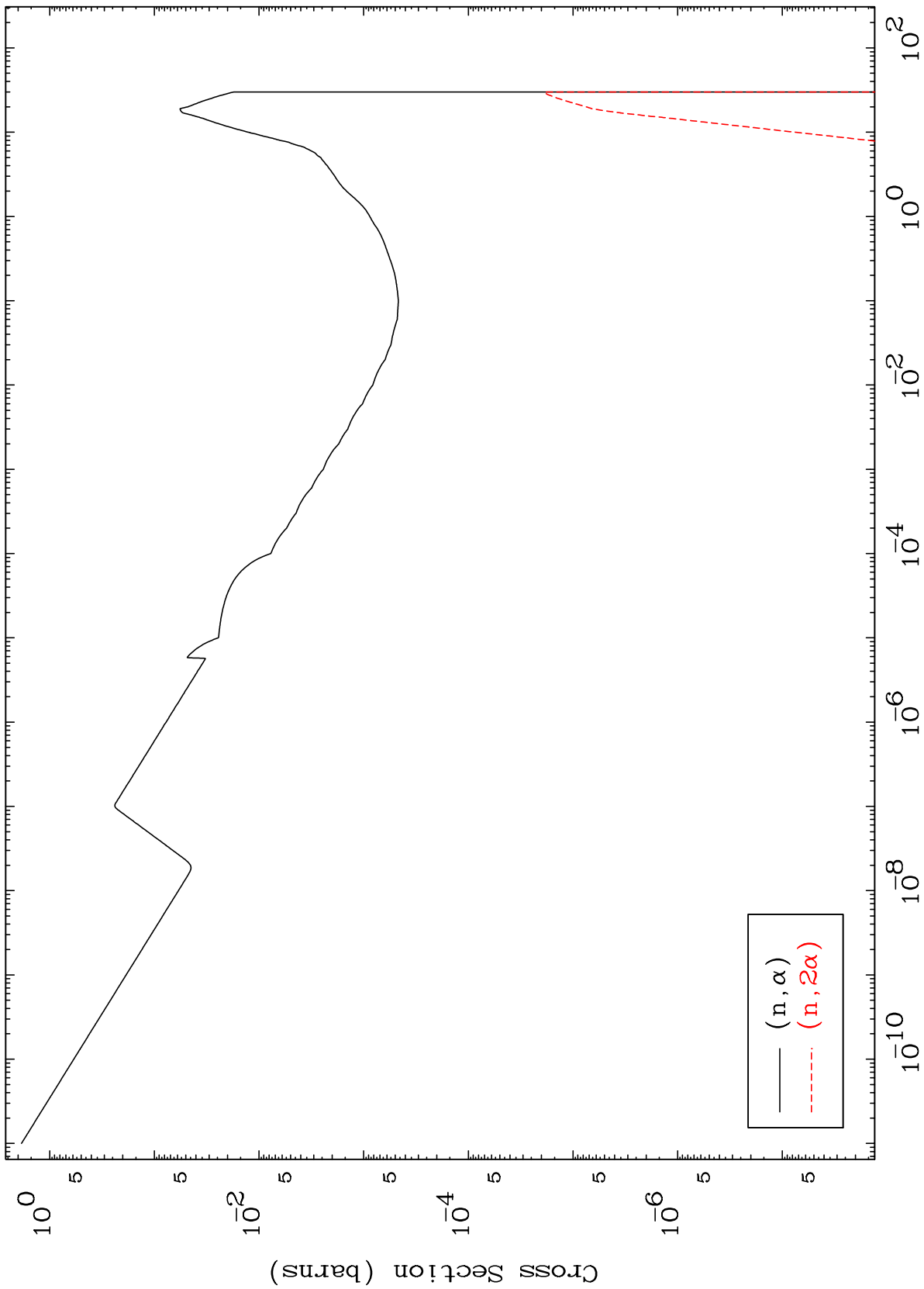
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(n, α) Levels
293 Kelvin Cross Sections



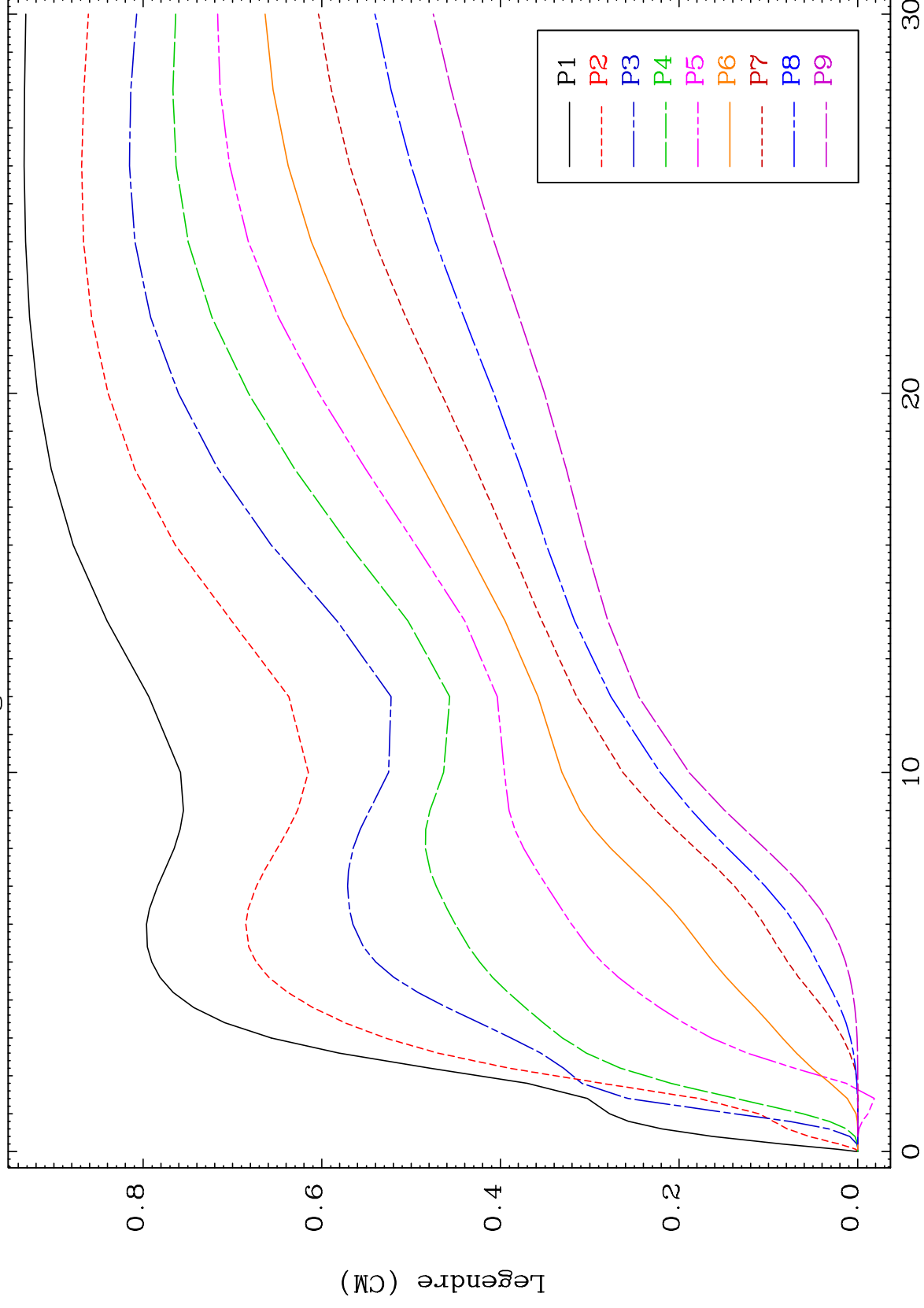
17

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Elastic Legendre Coefficients



18

0

10

20

Incident Energy (MeV)

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P1

P2

P3

P4

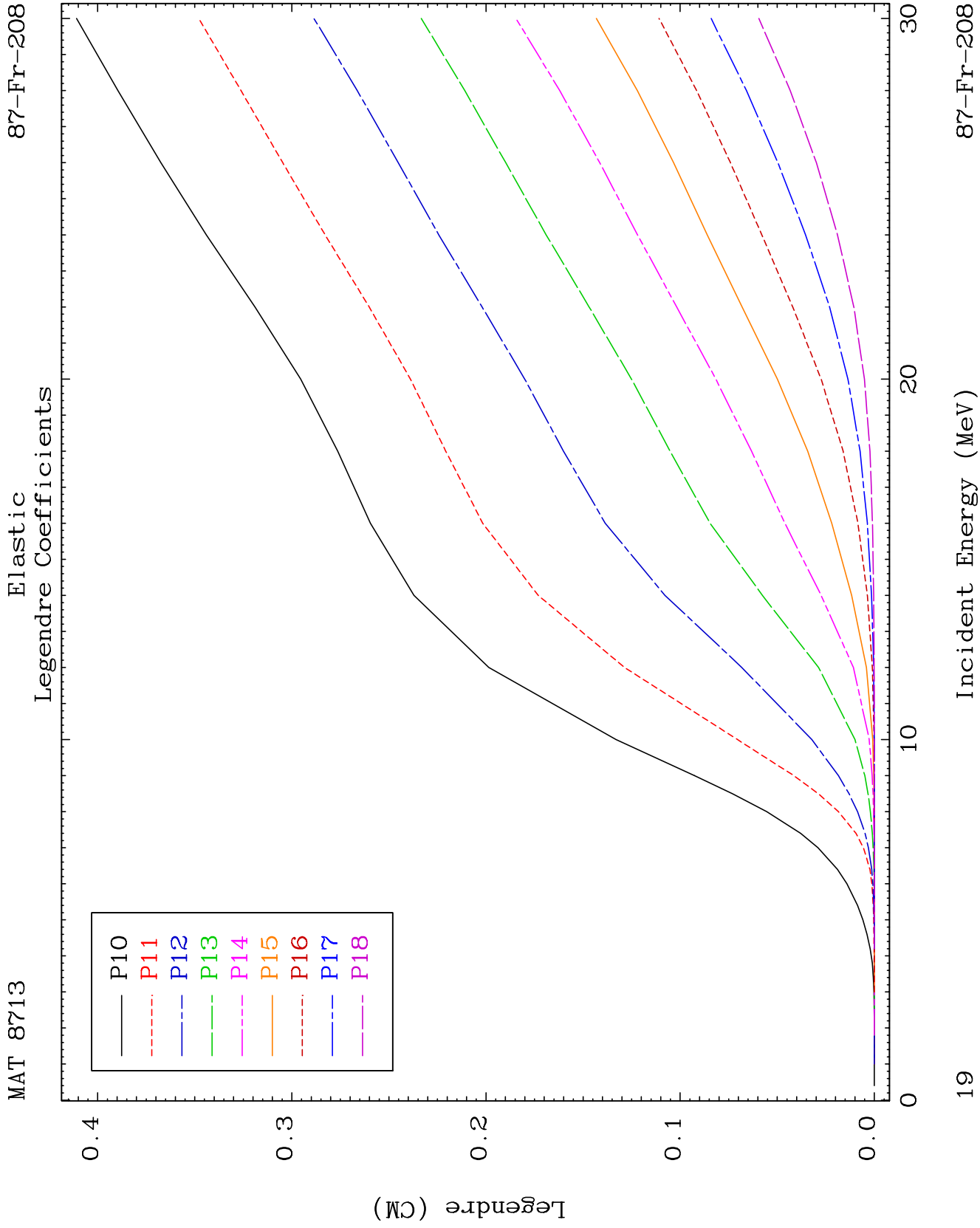
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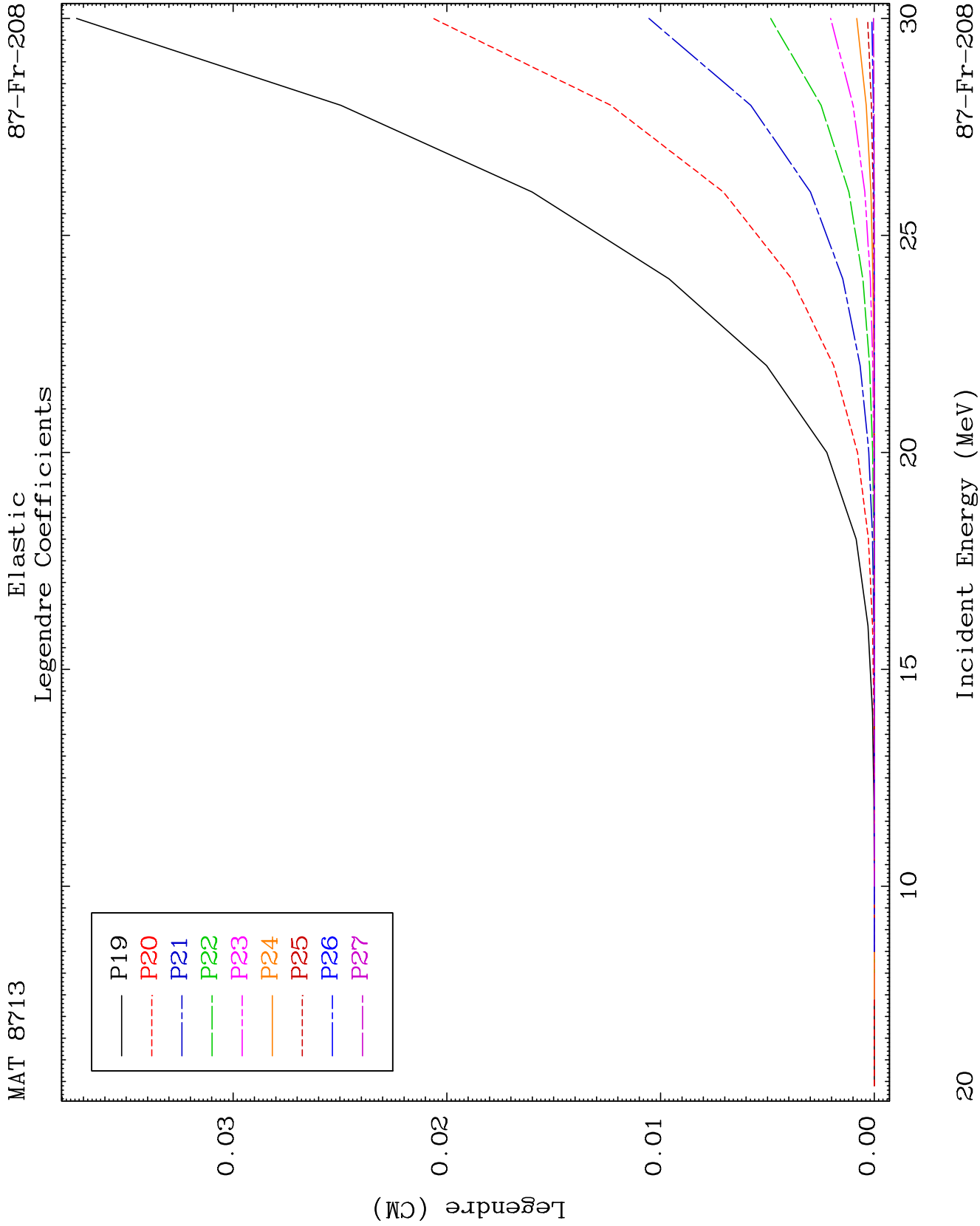
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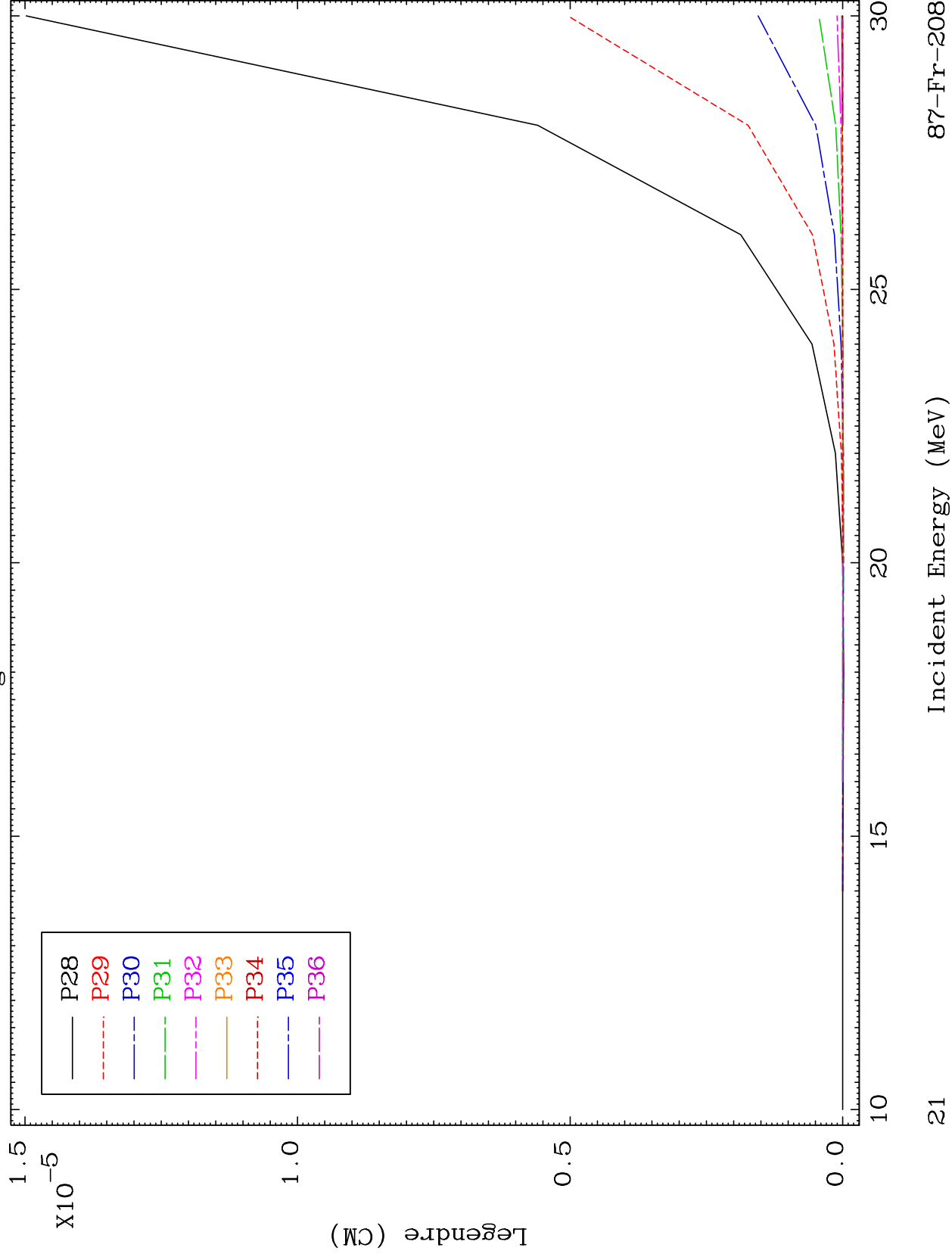


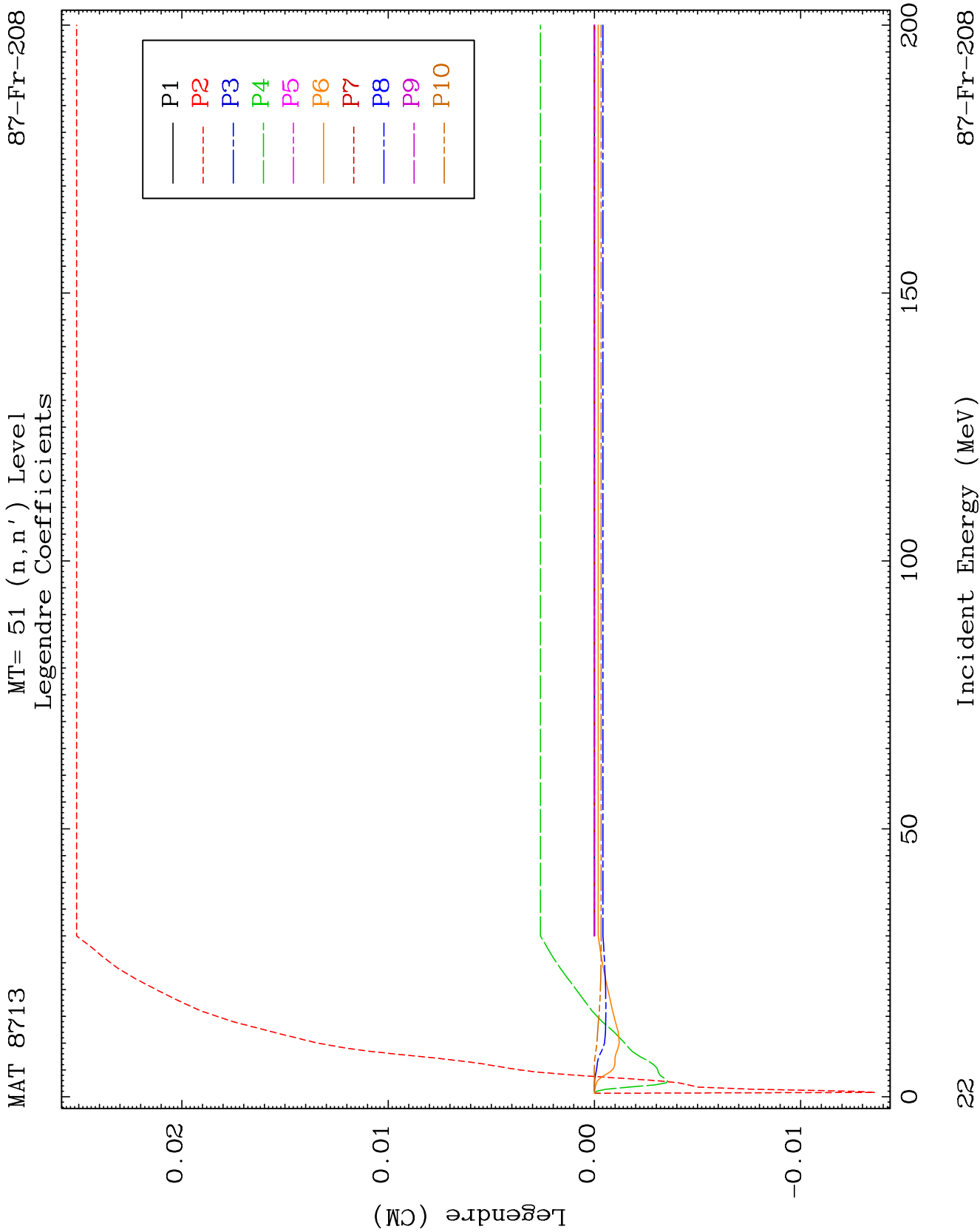


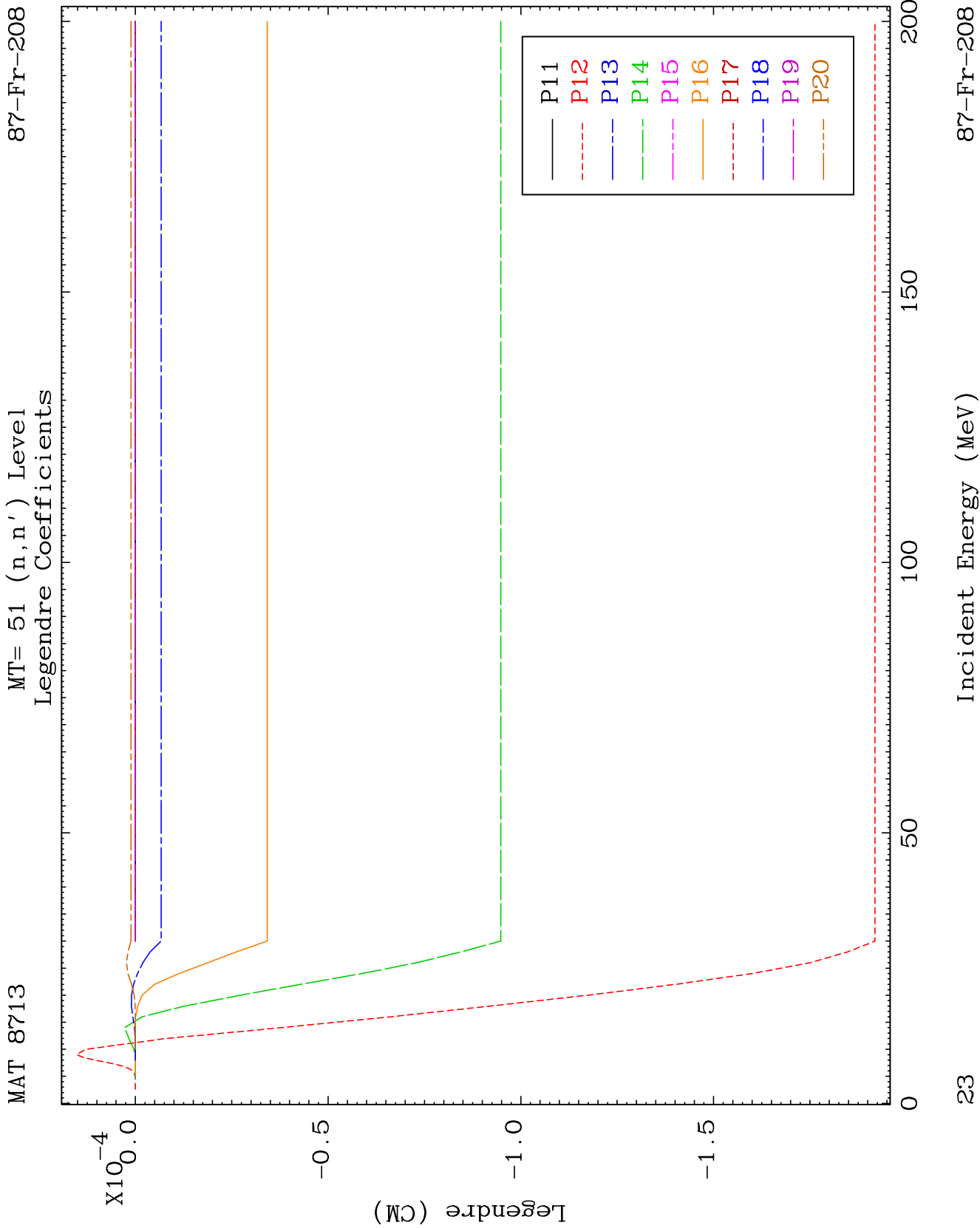
MAT 8713

Elastic Legendre Coefficients

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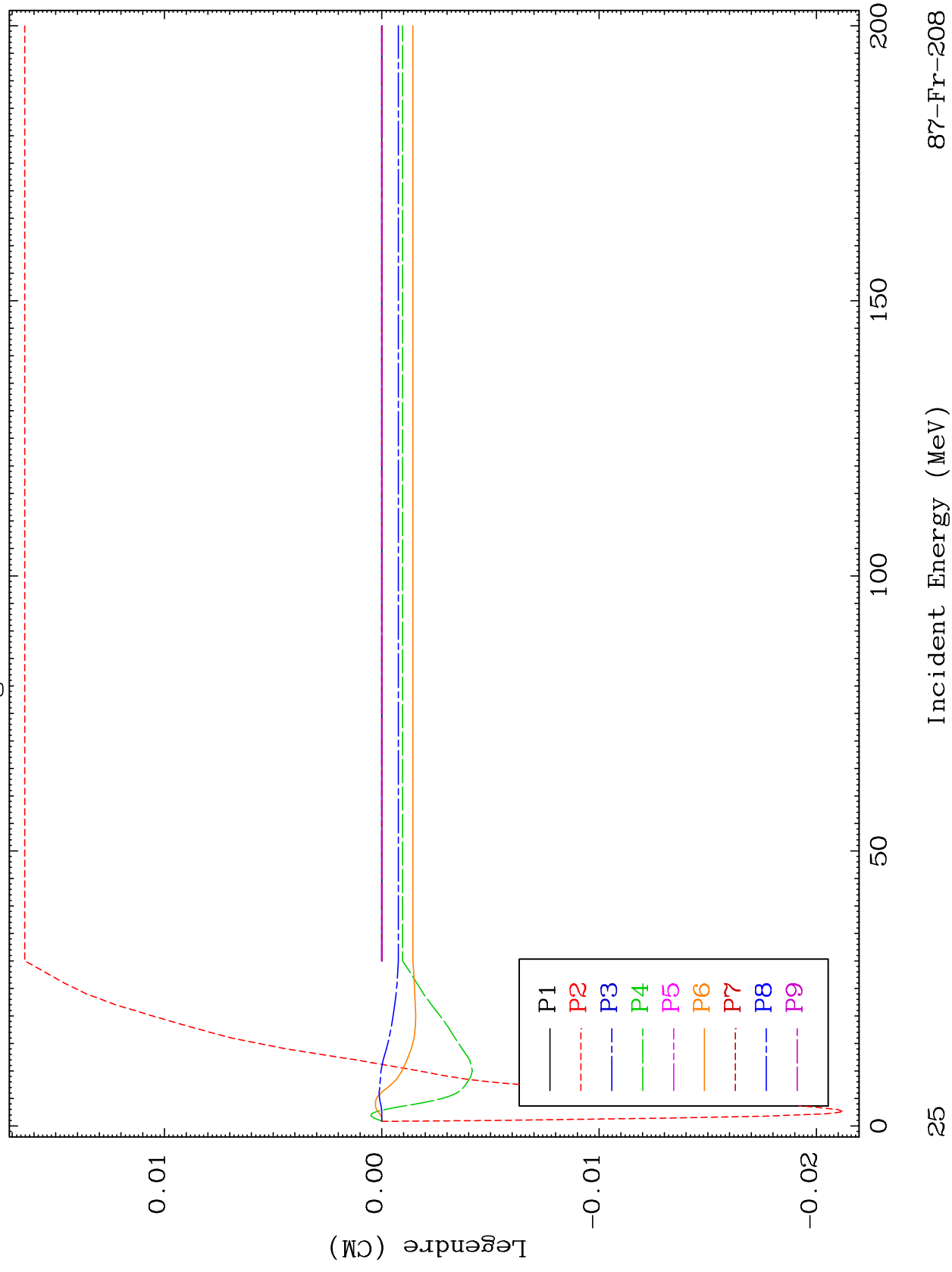


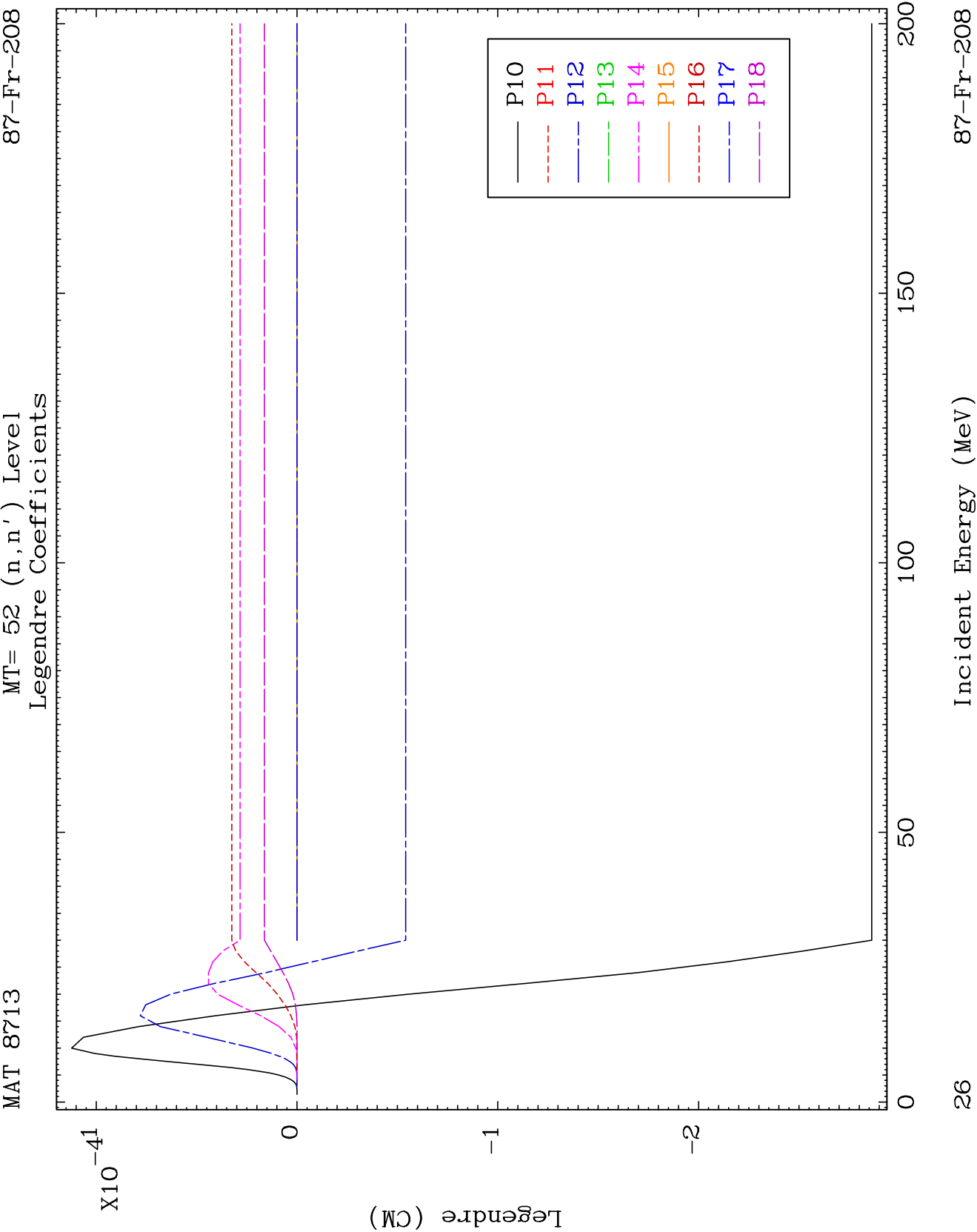


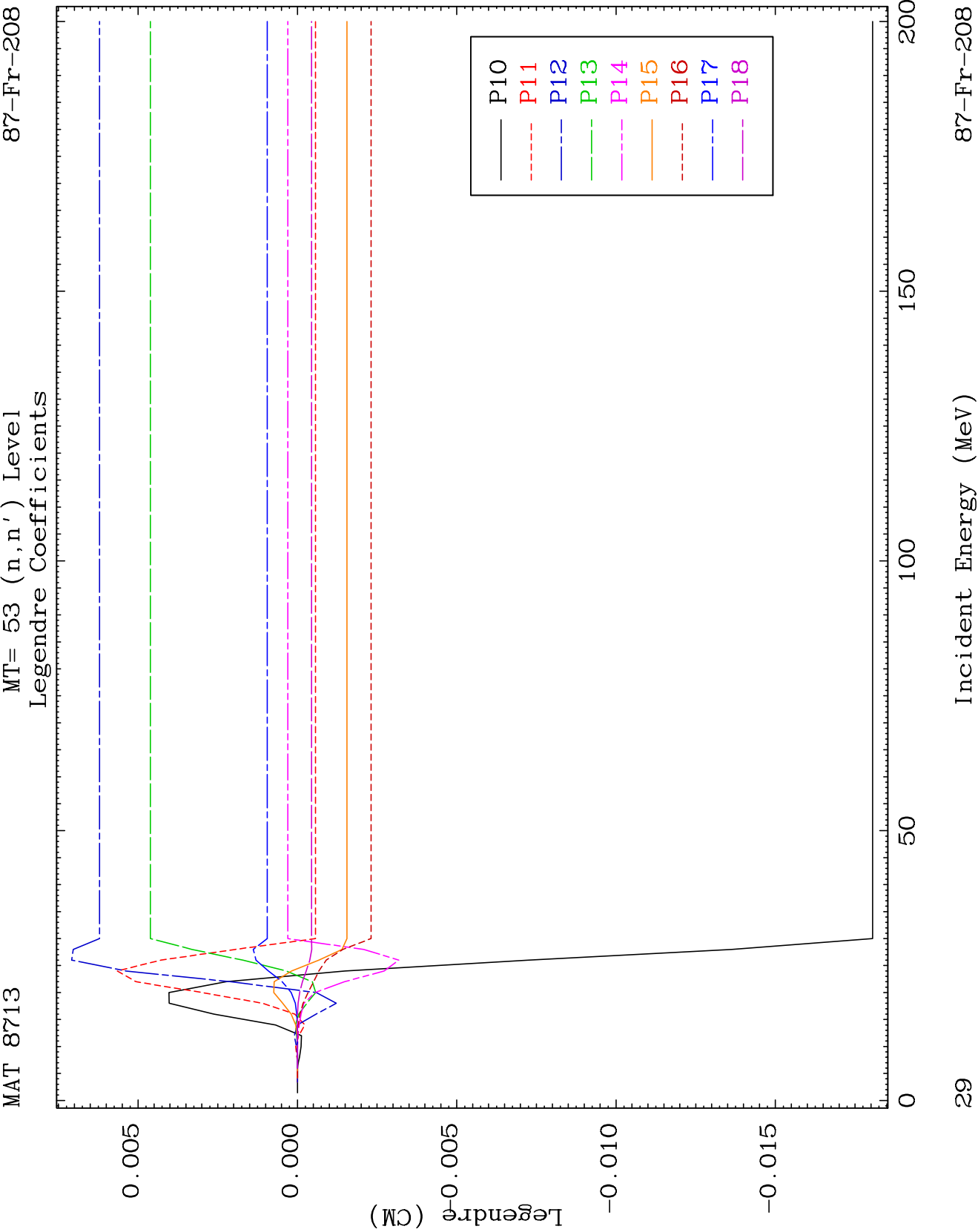
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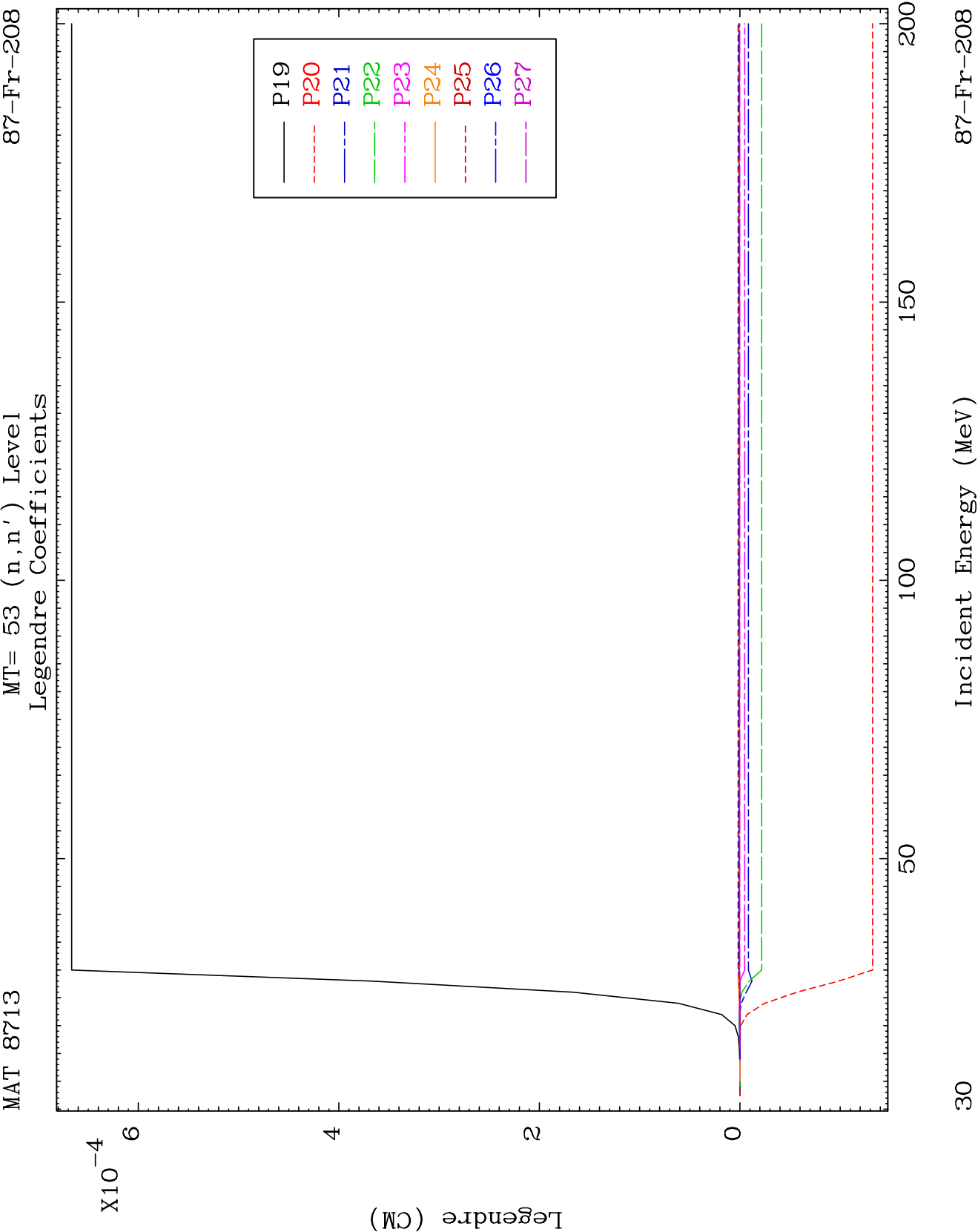
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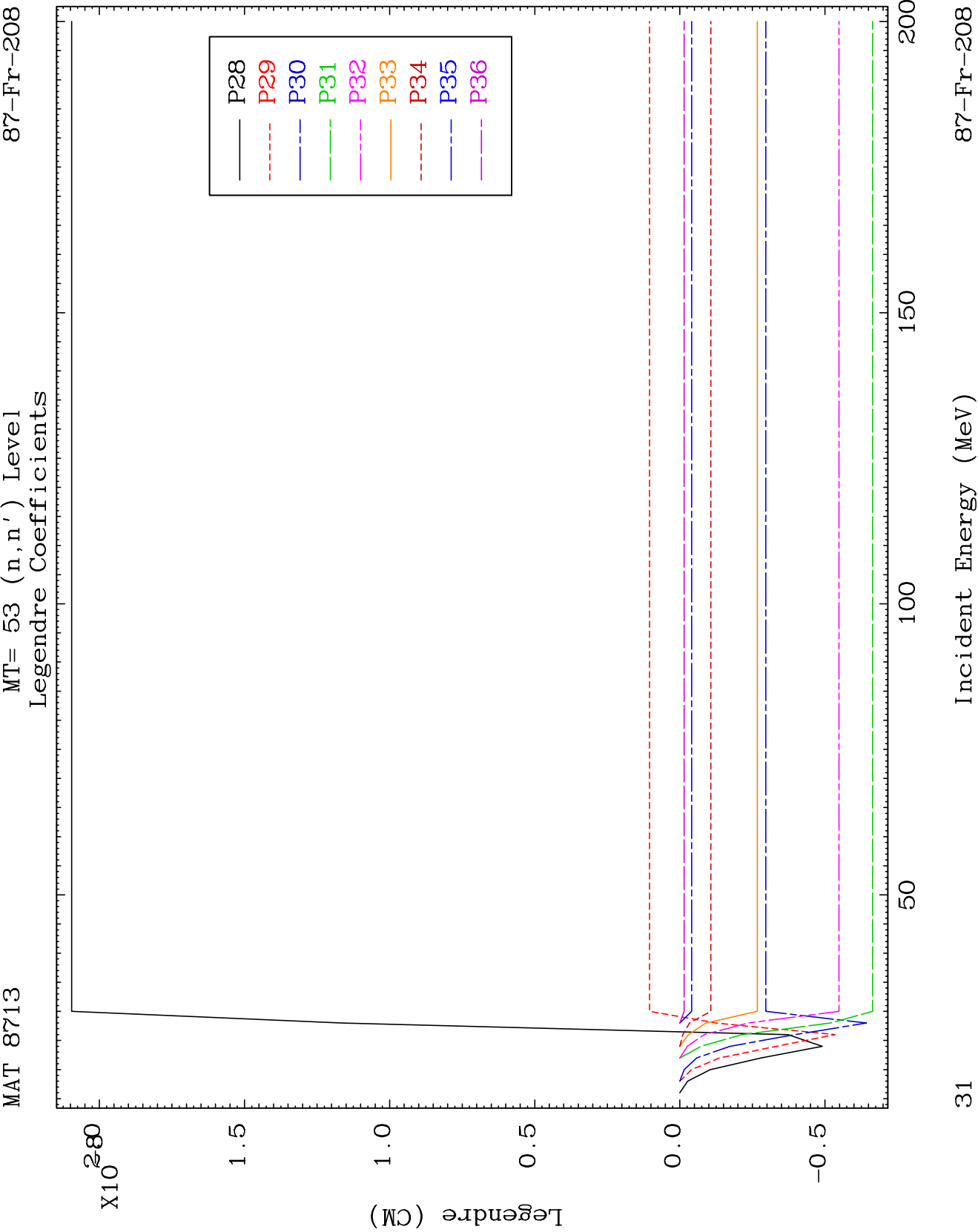
MT= 52 (n, n') Level
Legendre Coefficients

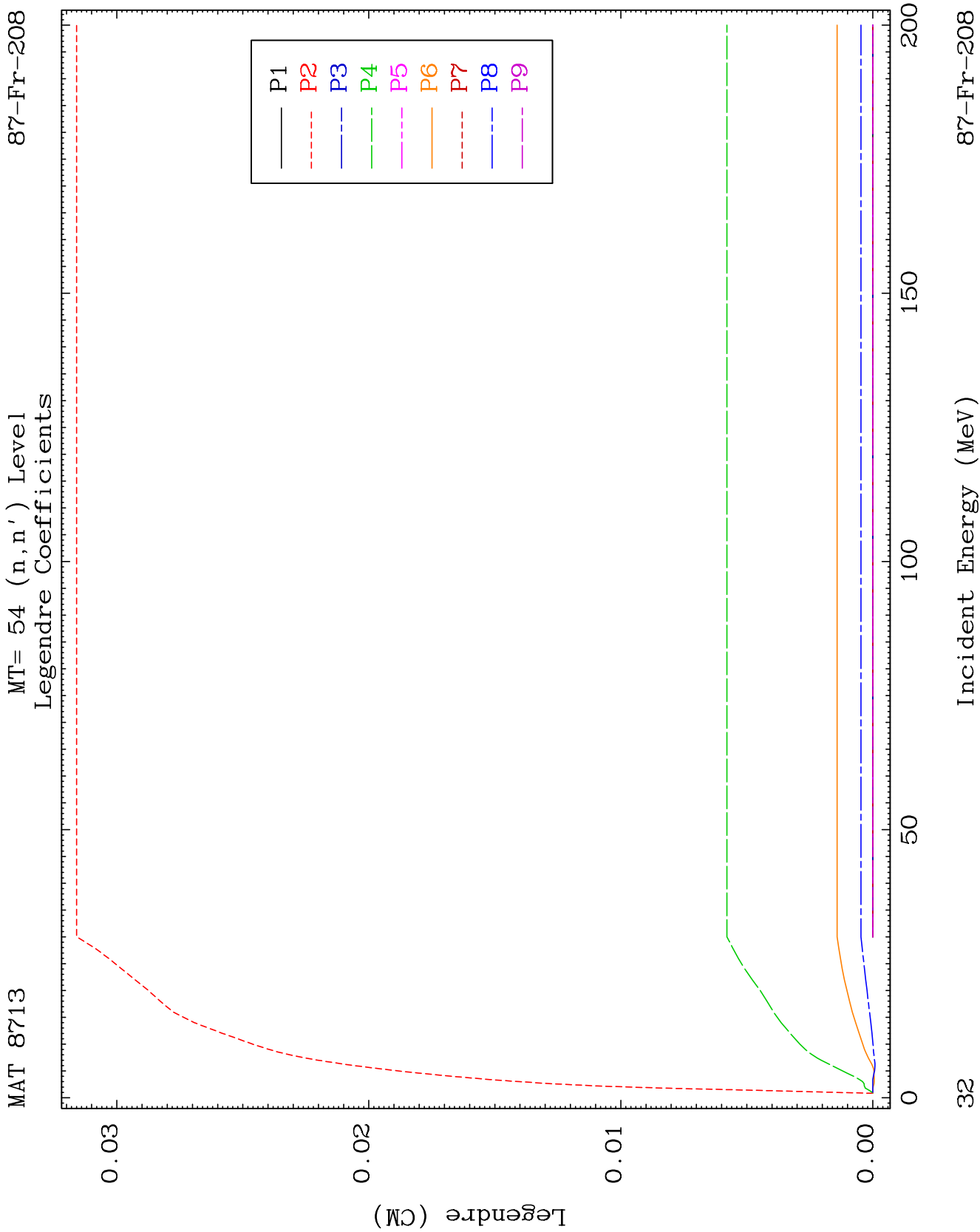


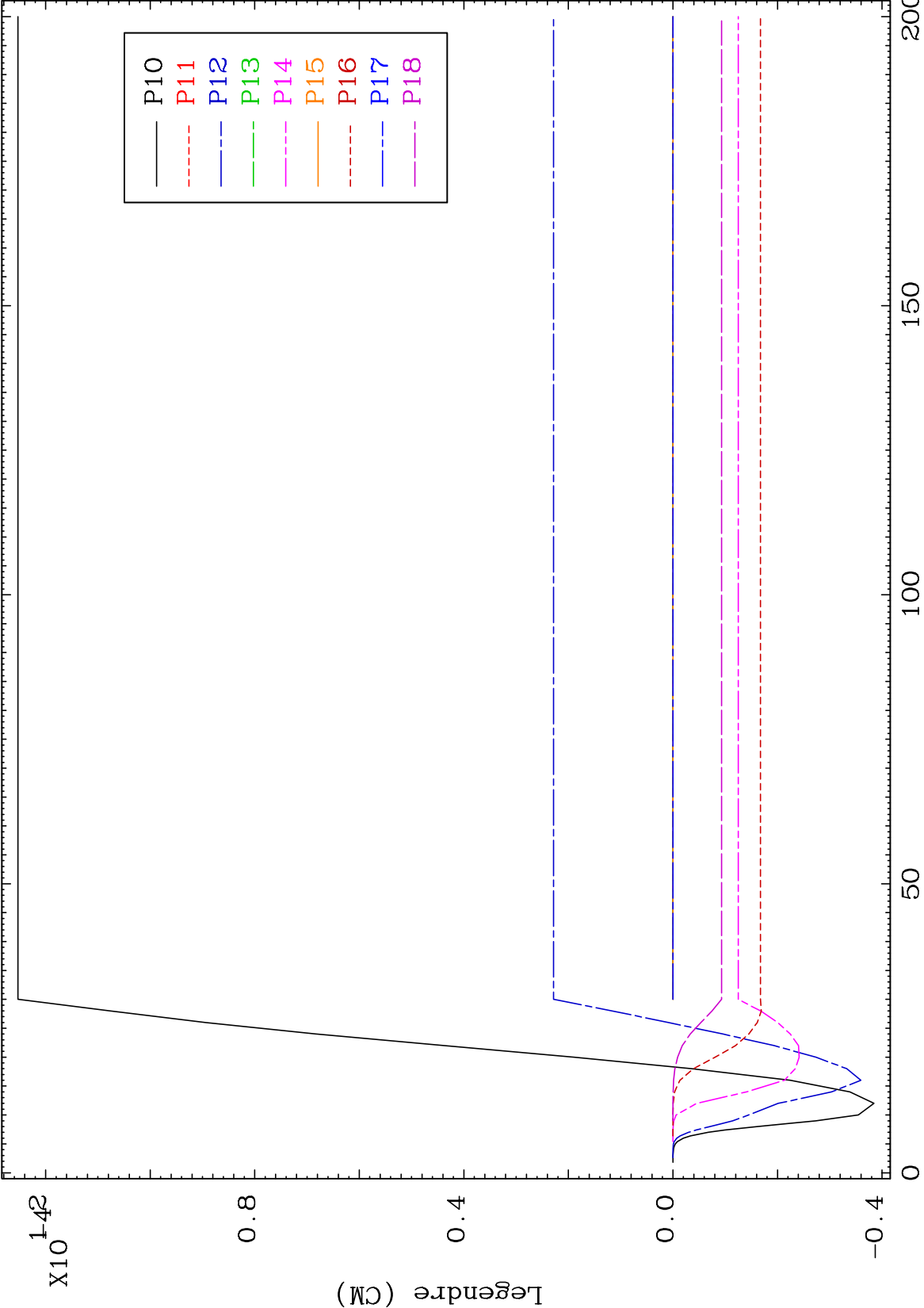


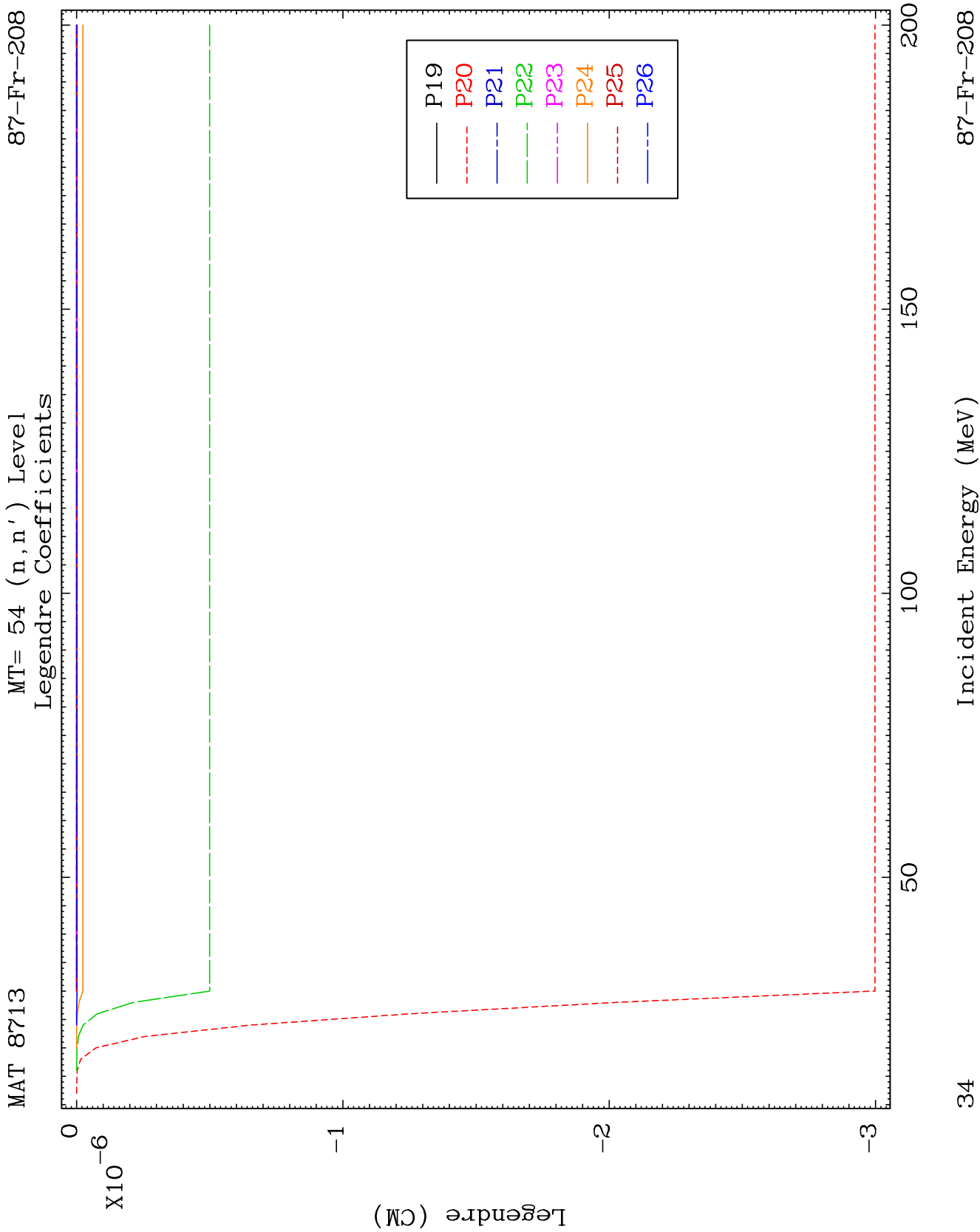








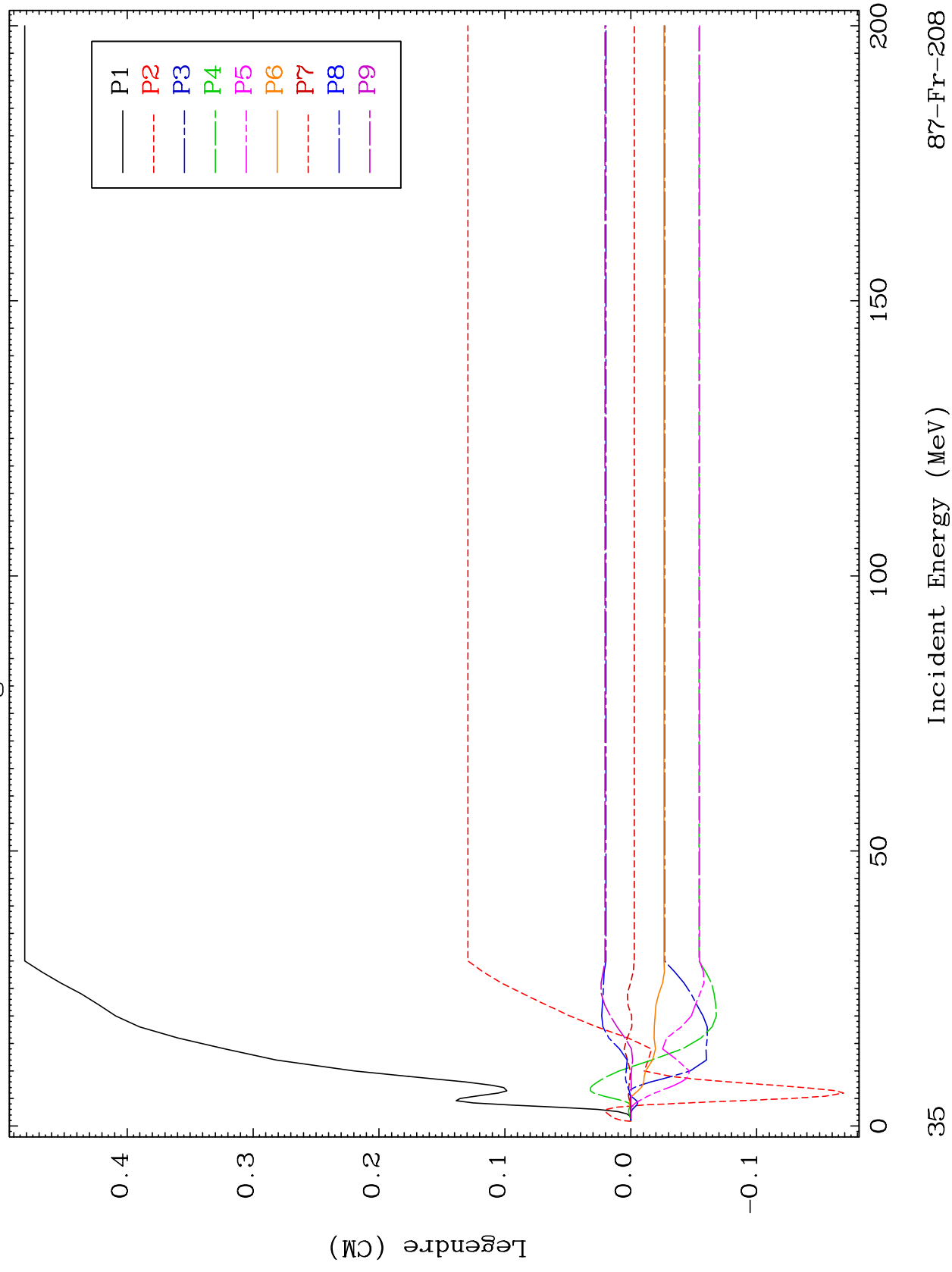




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MT= 55 (n, n') Level
Legendre Coefficients

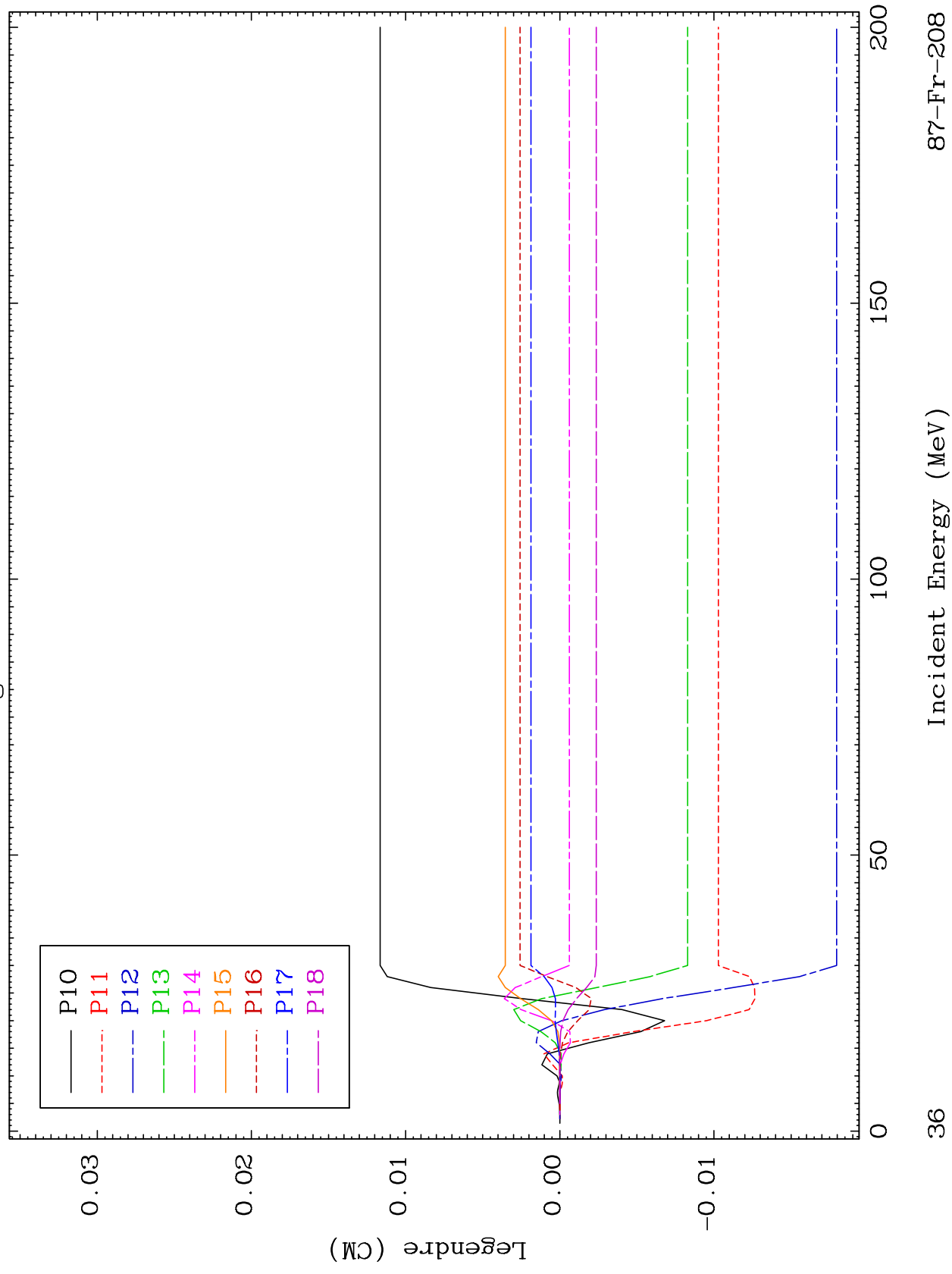
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MAT 8713

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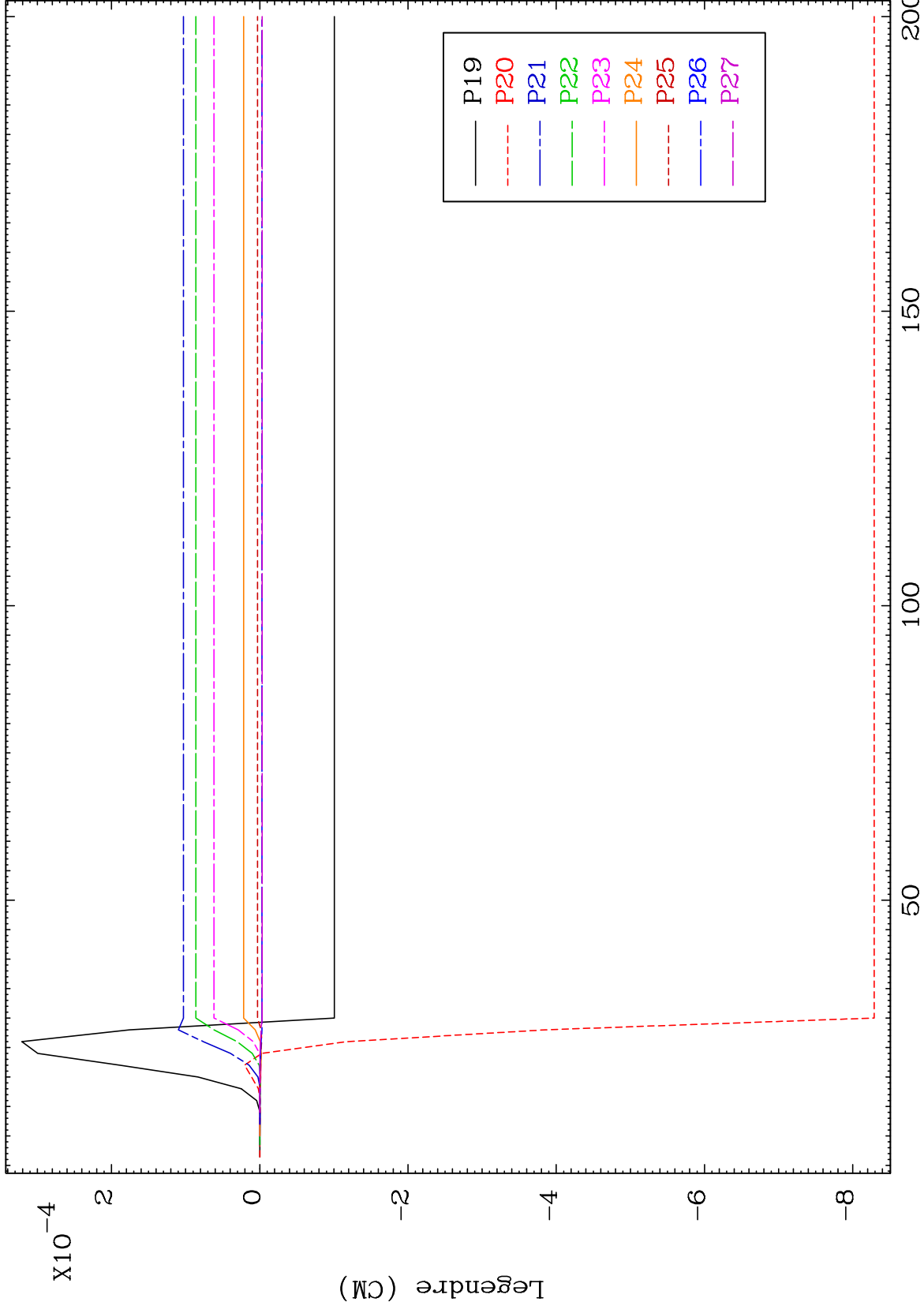
MT= 55 (n, n') Level
Legendre Coefficients



MAT 8713

MT= 55 (n,n') Level
Legendre Coefficients

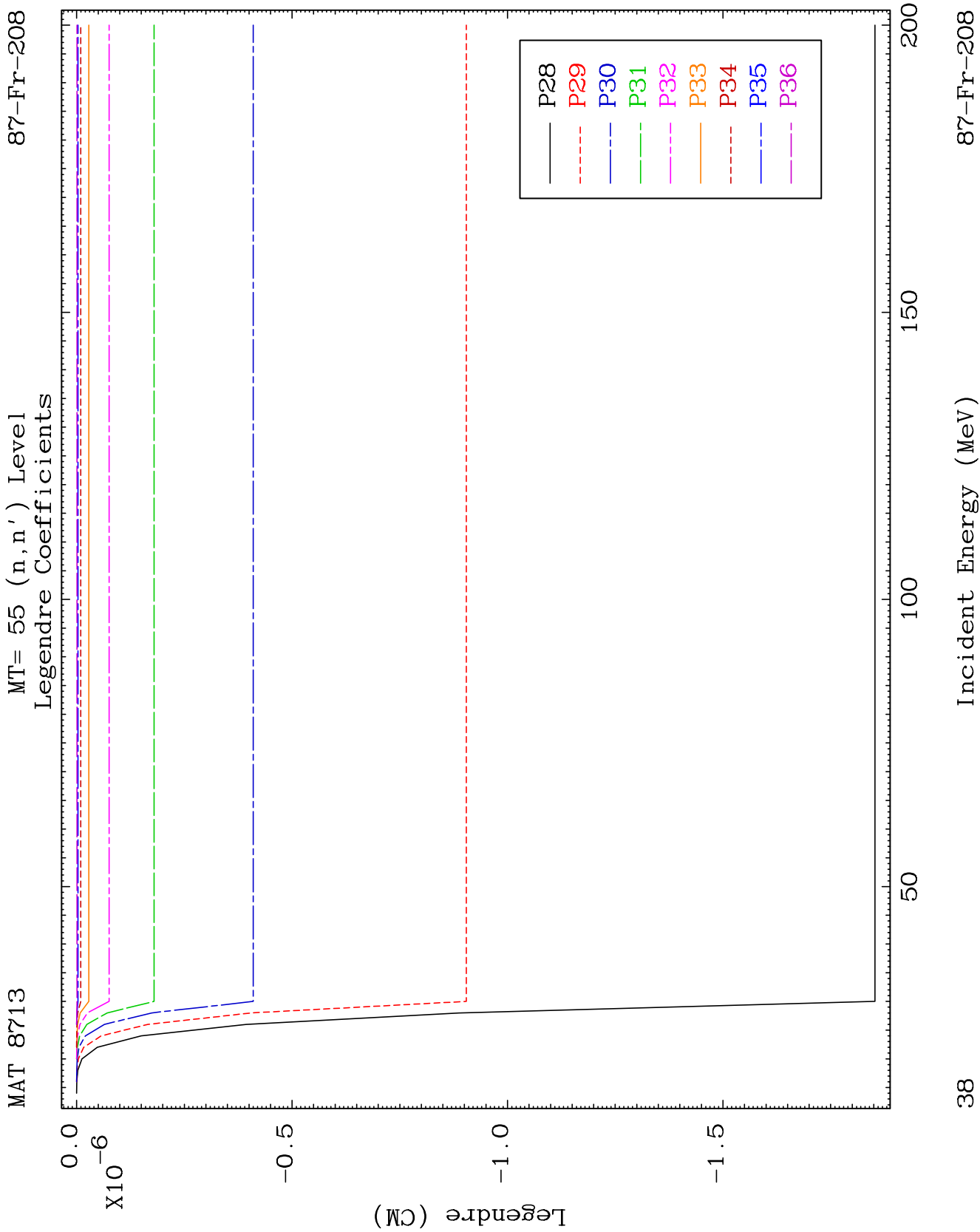
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Incident Energy (MeV)

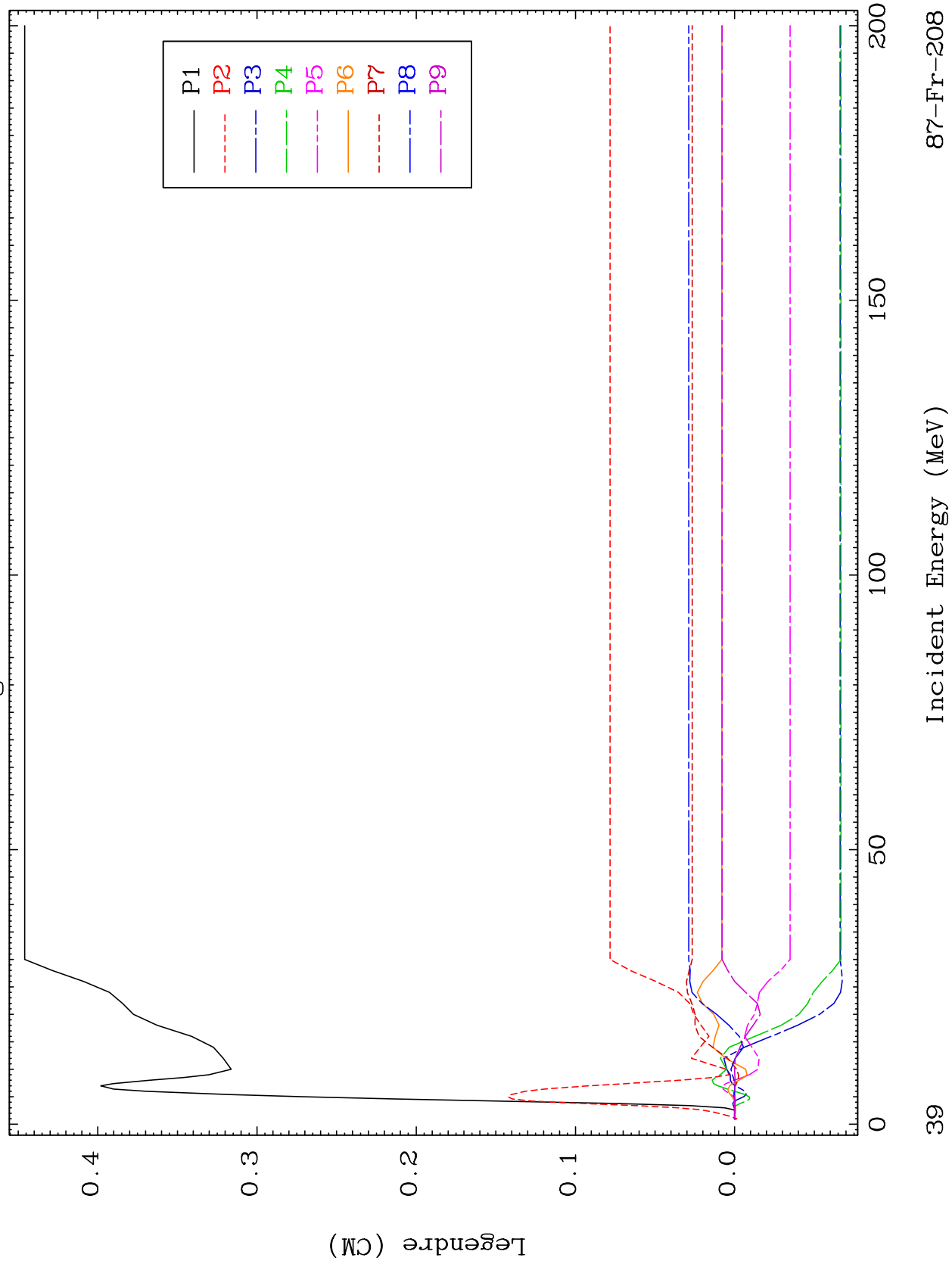
87-Fr-208



MAT 8713

MT= 56 (n, n') Level
Legendre Coefficients

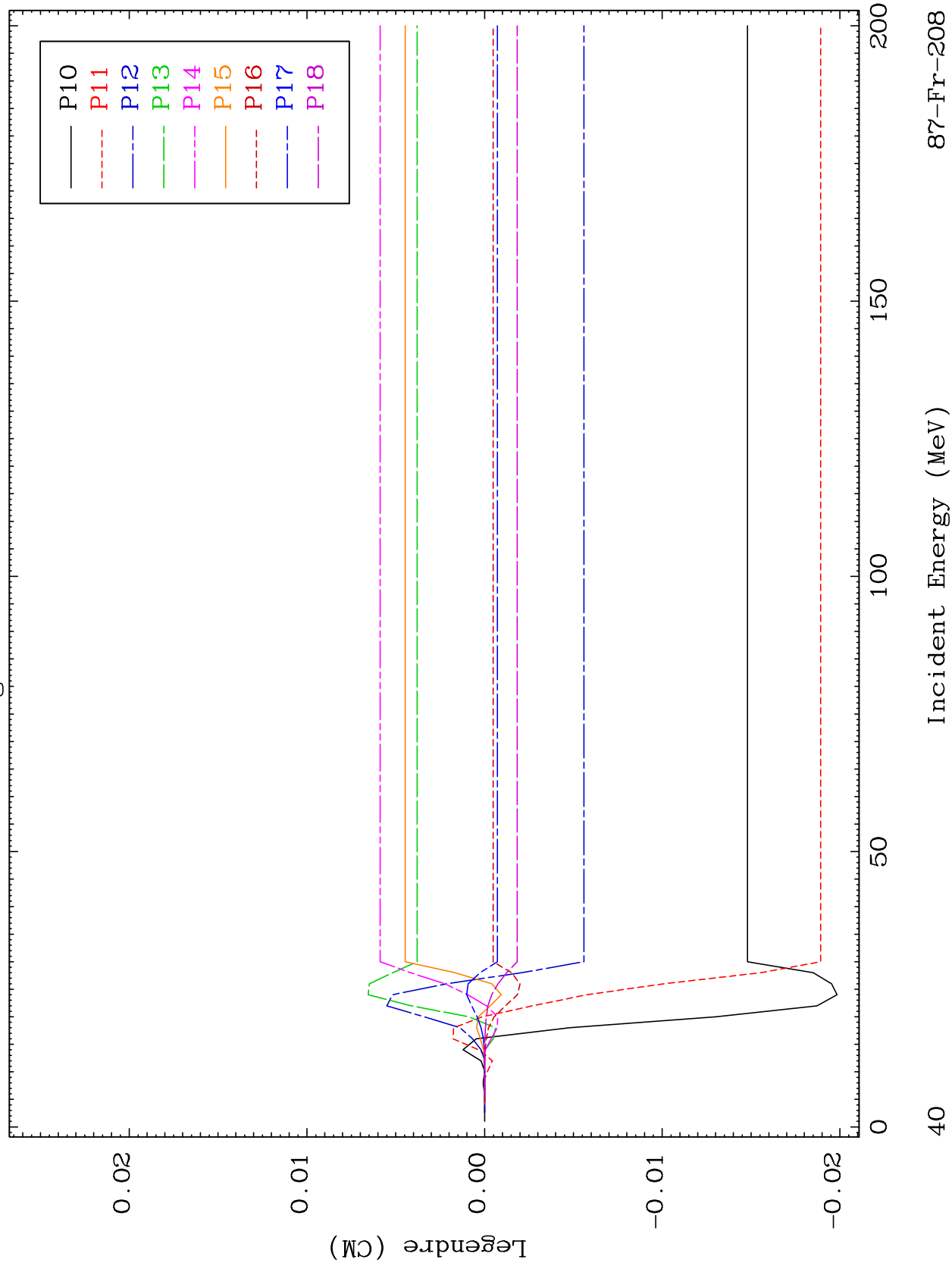
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MAT 8713

MT= 56 (n, n') Level
Legendre Coefficients

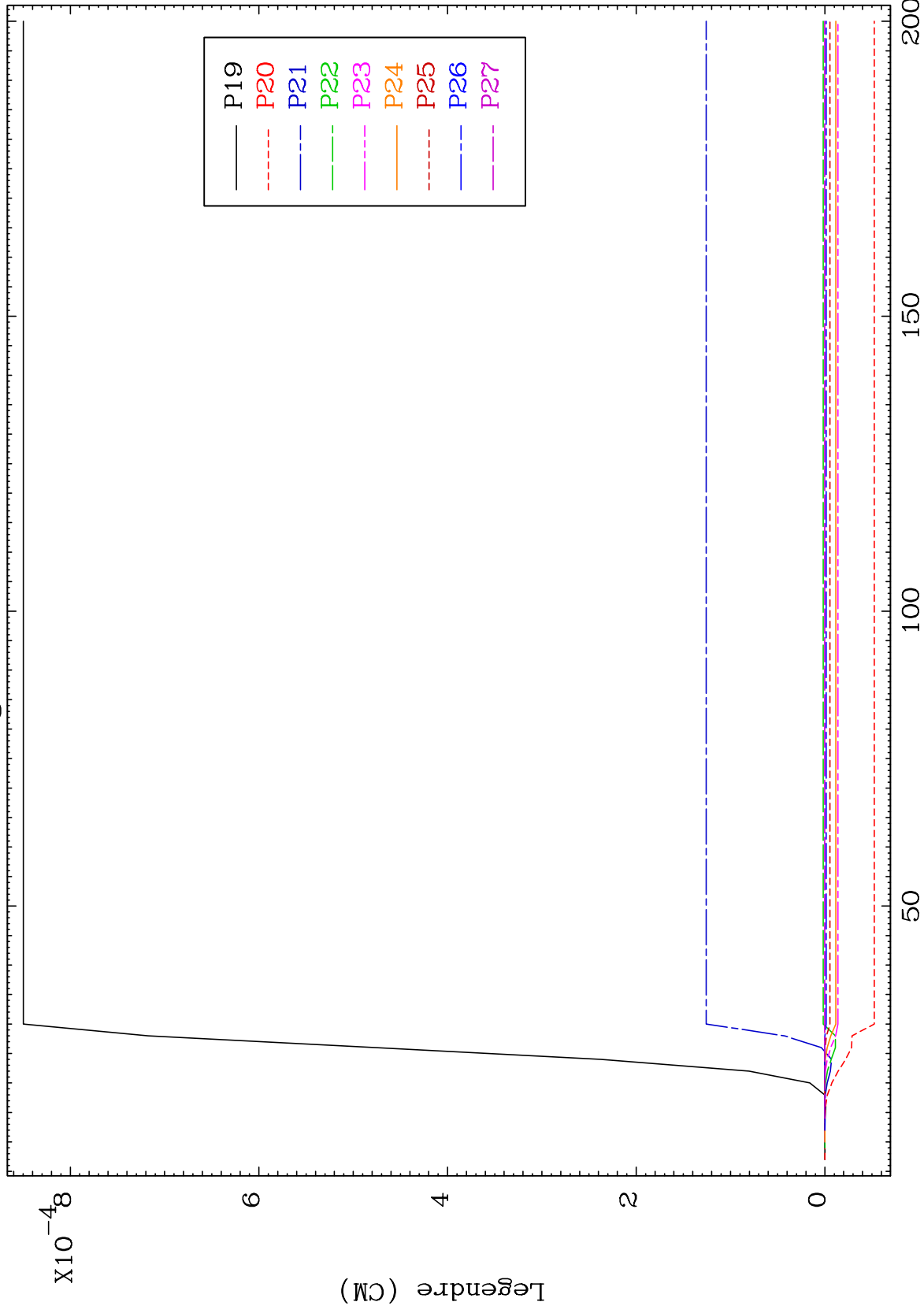
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MAT 8713

MT= 56 (n,n') Level
Legendre Coefficients

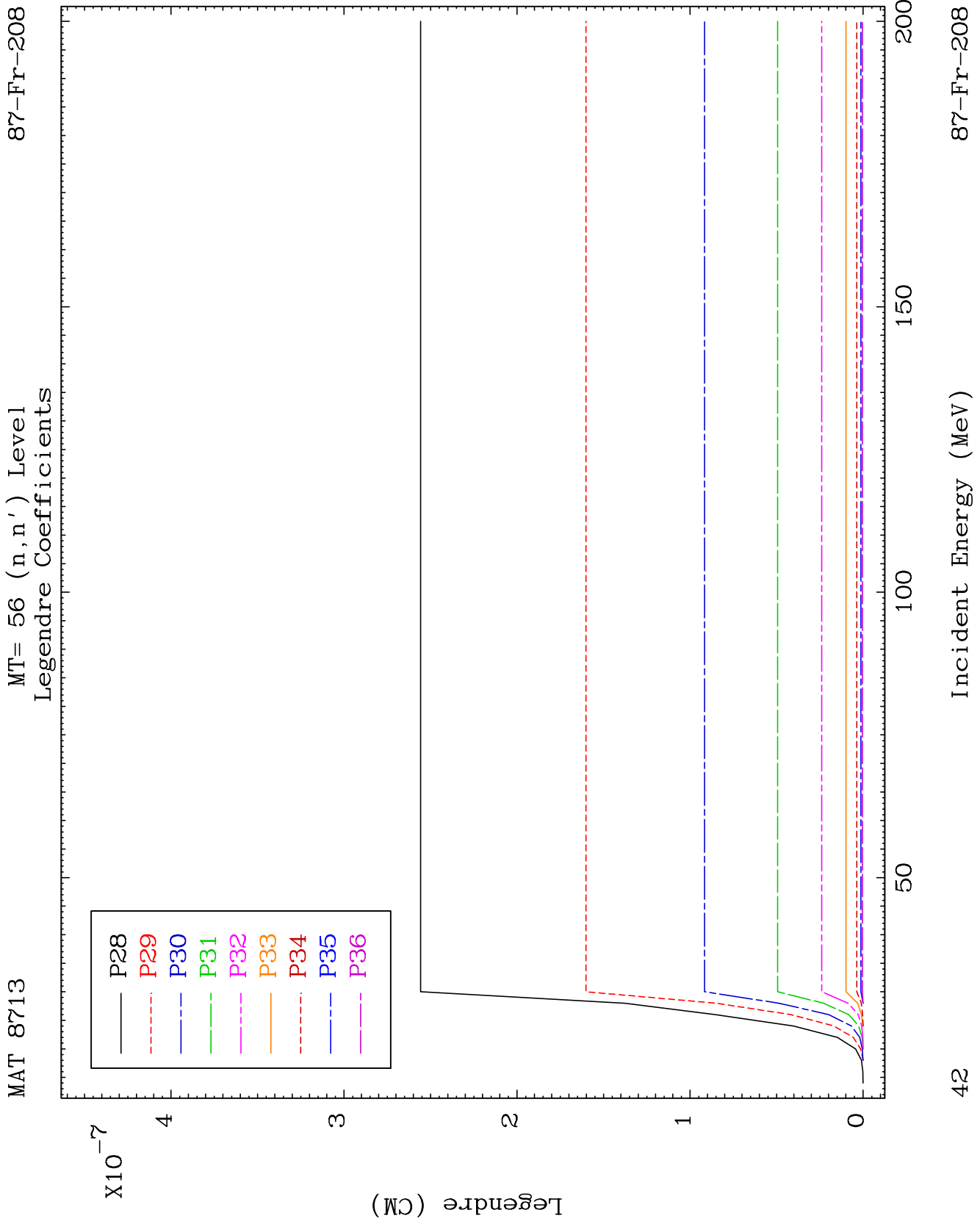
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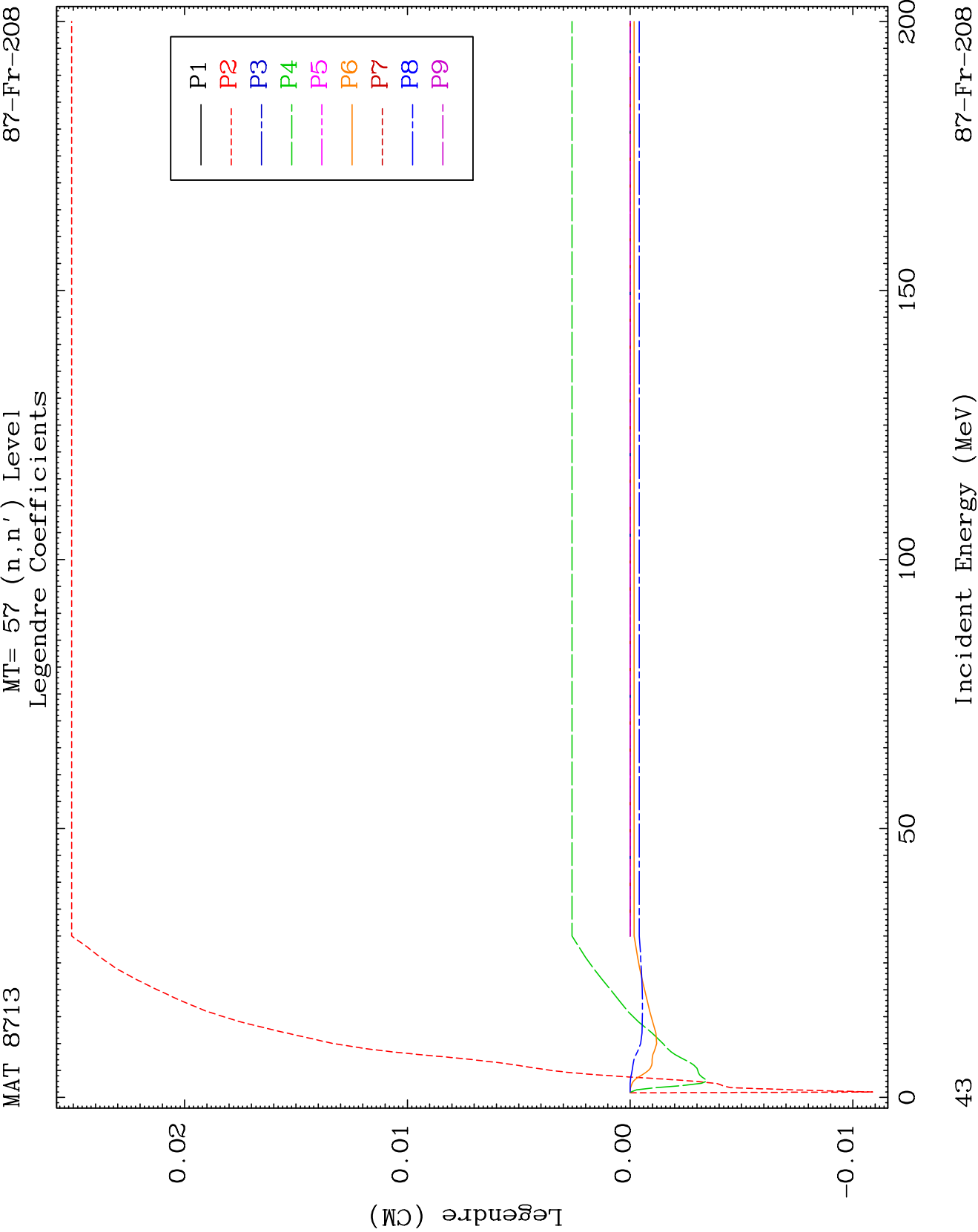


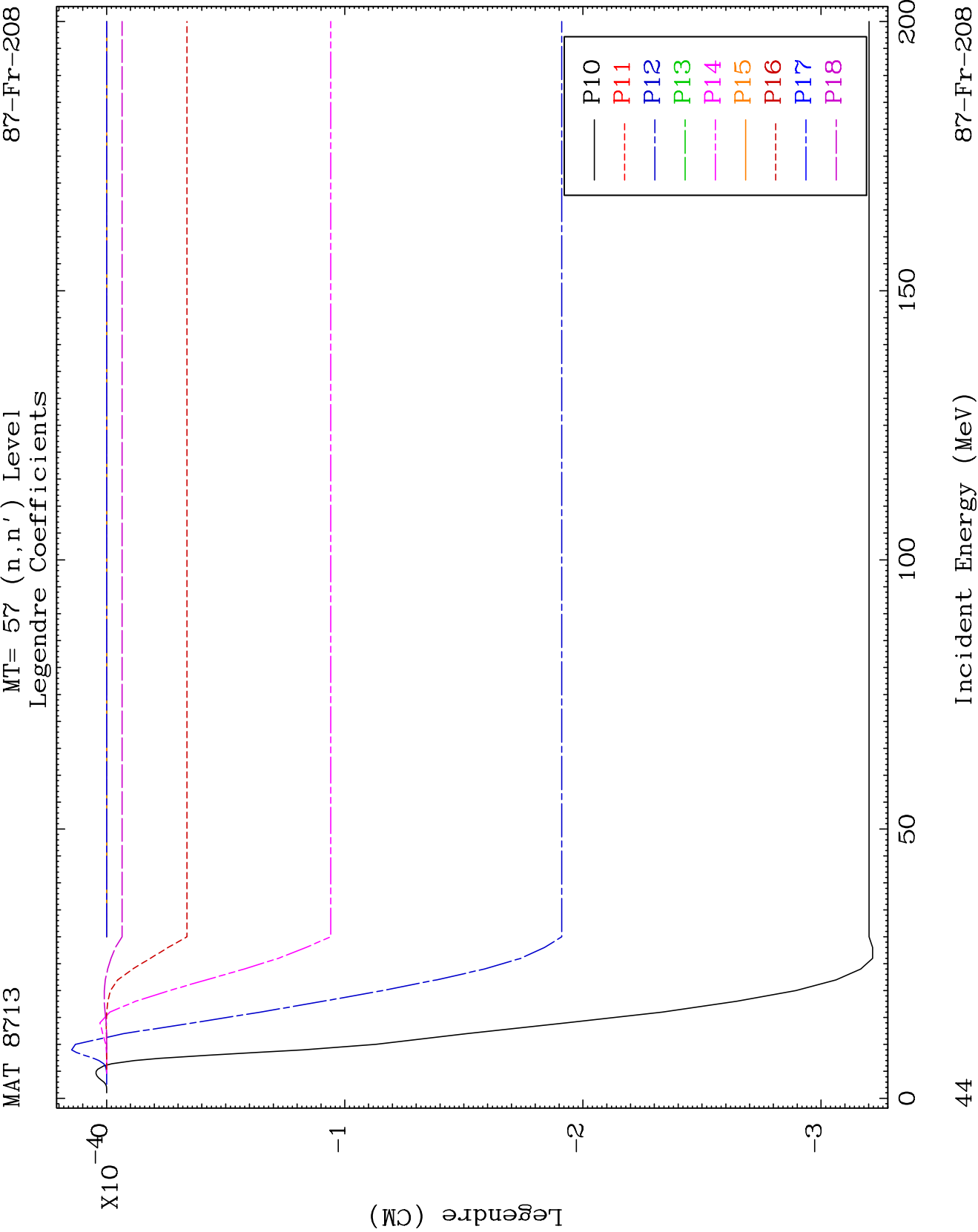
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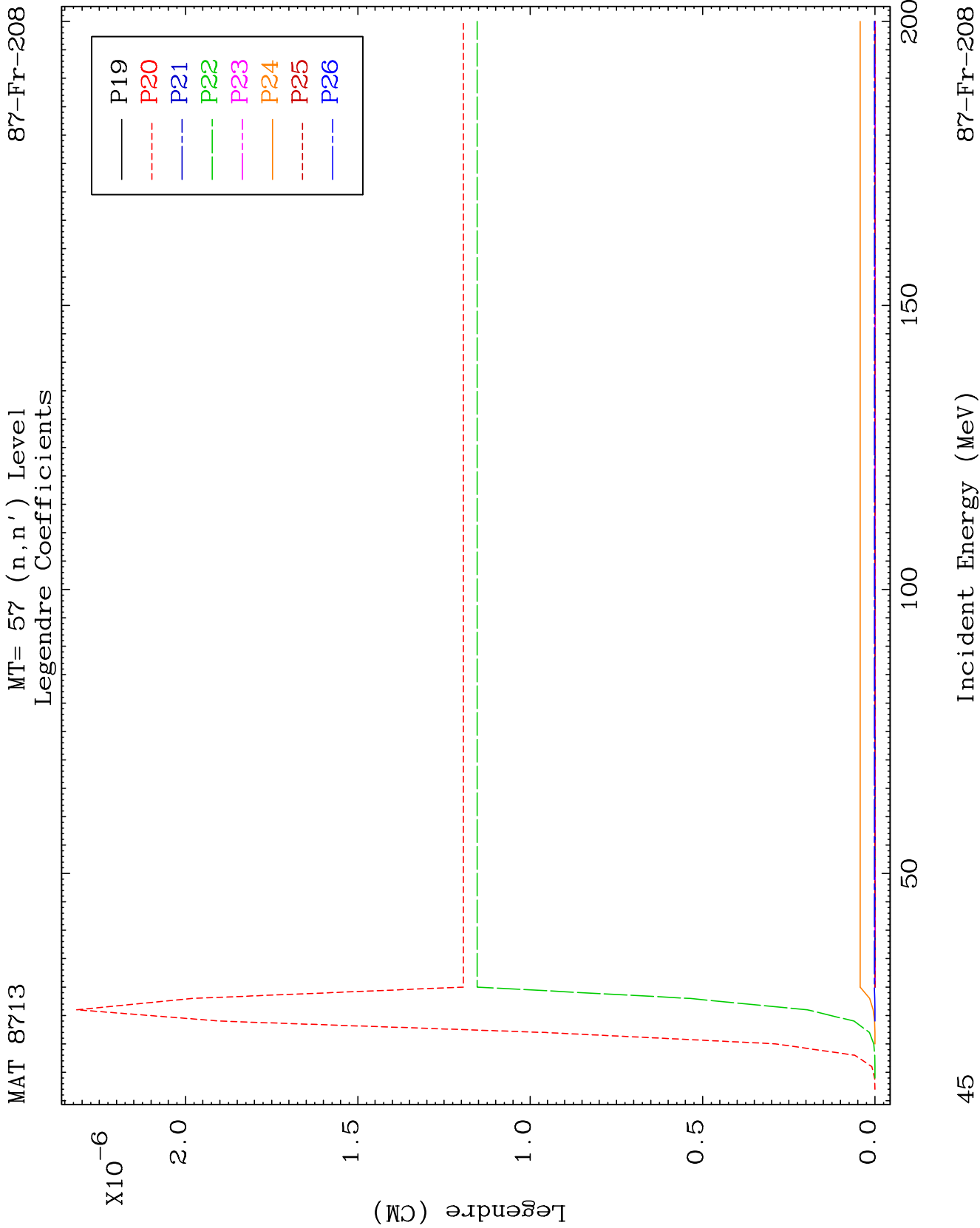
Incident Energy (MeV)

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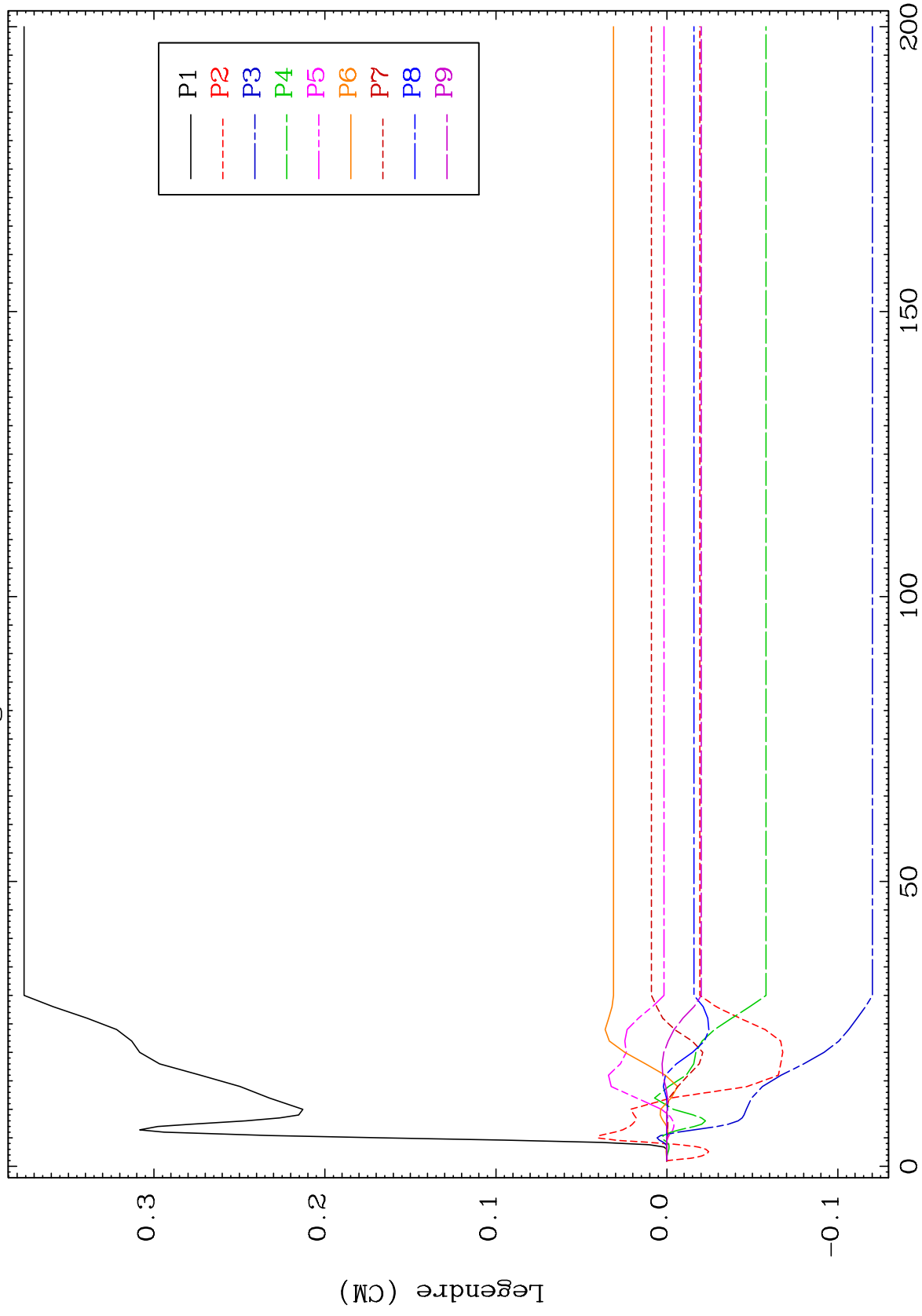




MAT 8713

MT= 58 (n,n') Level
Legendre Coefficients

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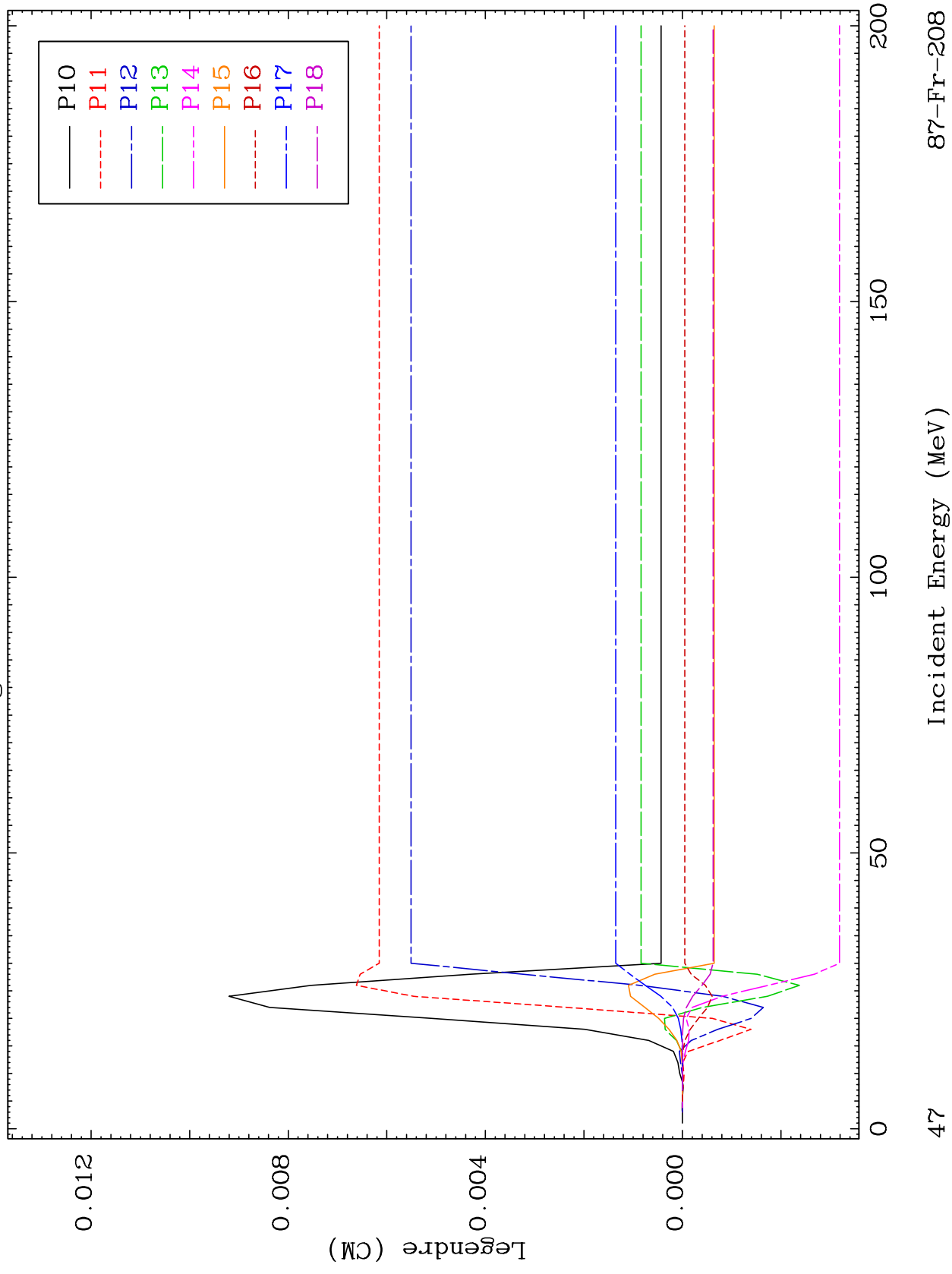
Incident Energy (MeV)

46

MAT 8713

MT= 58 (n, n') Level
Legendre Coefficients

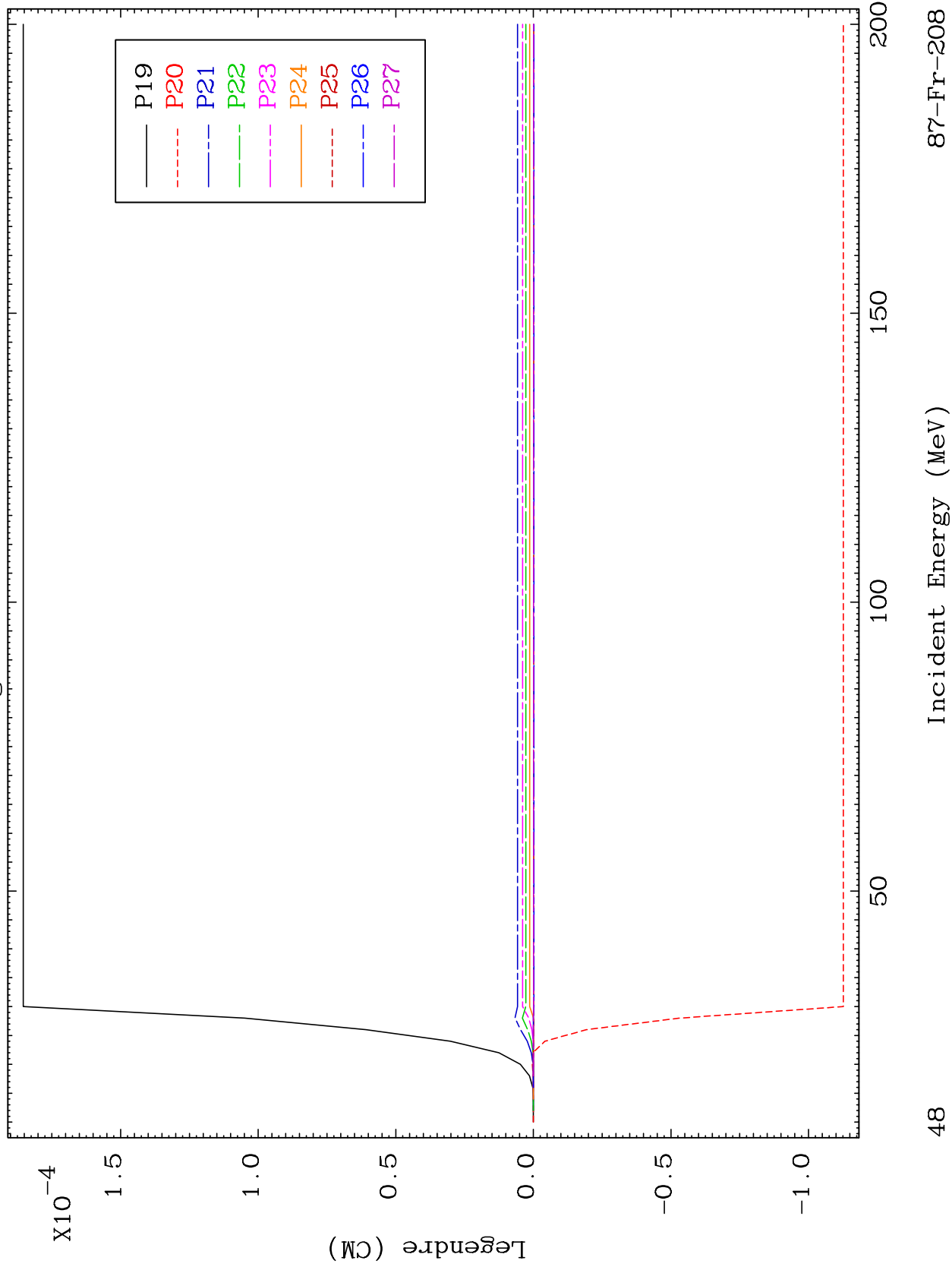
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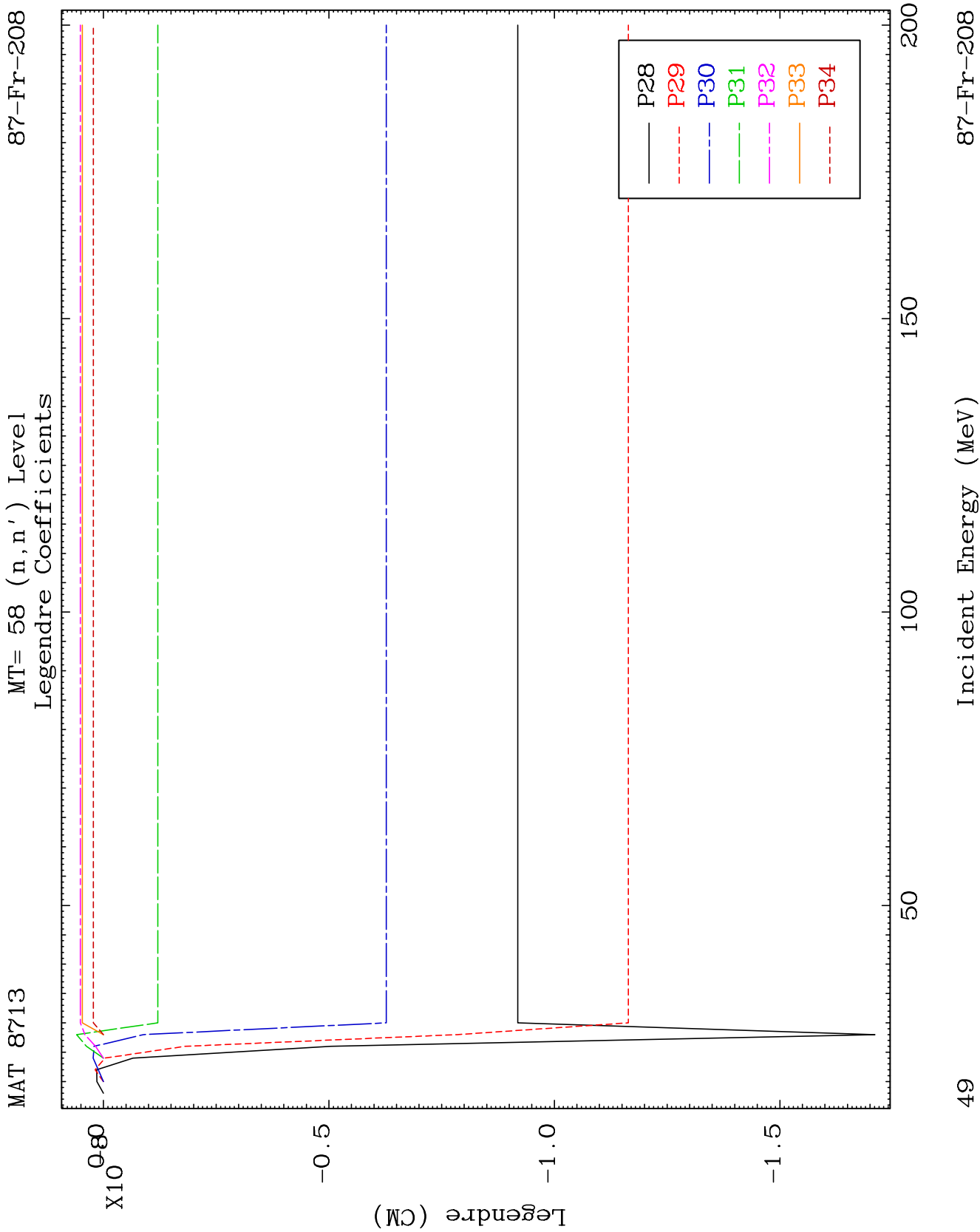


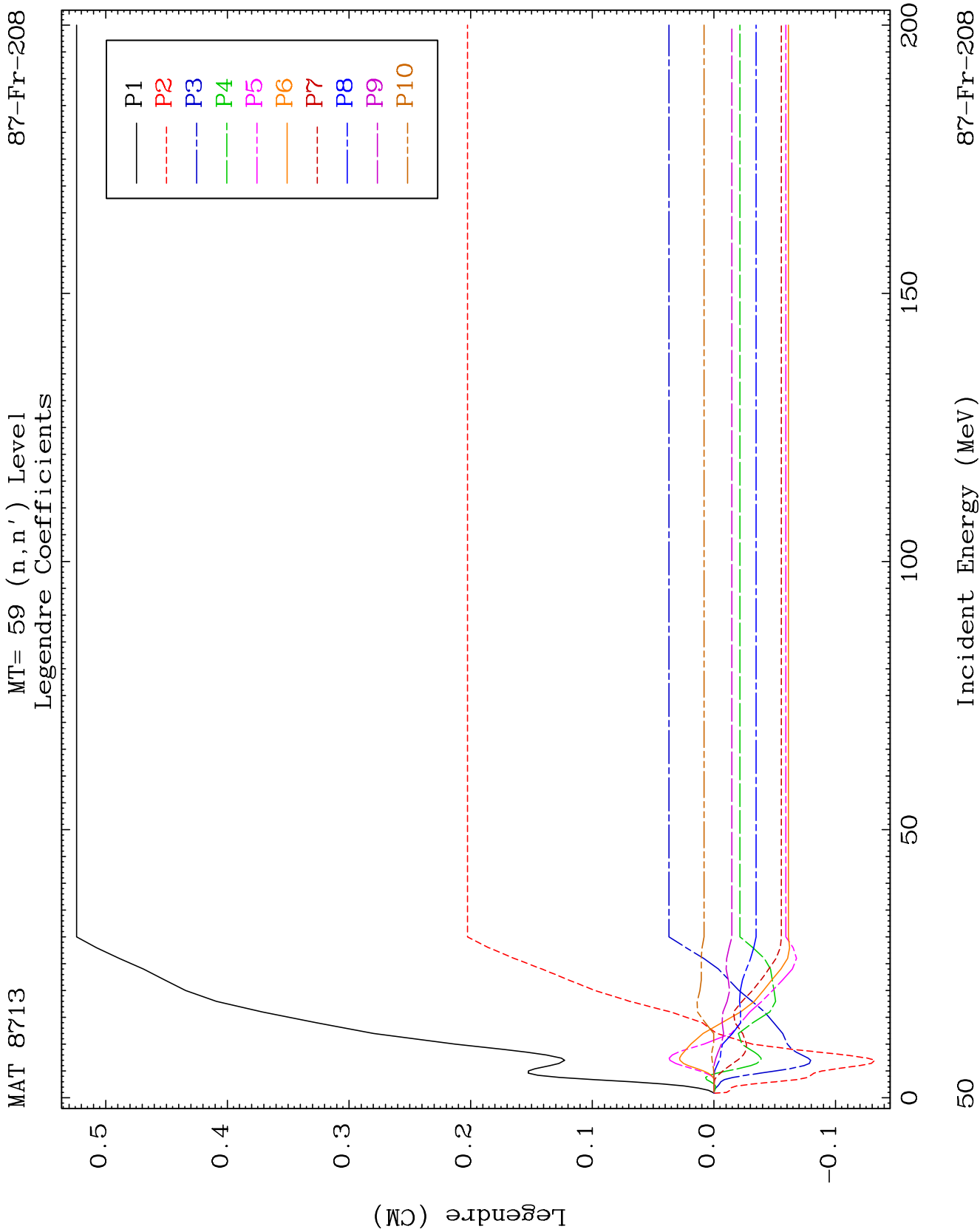
MAT 8713

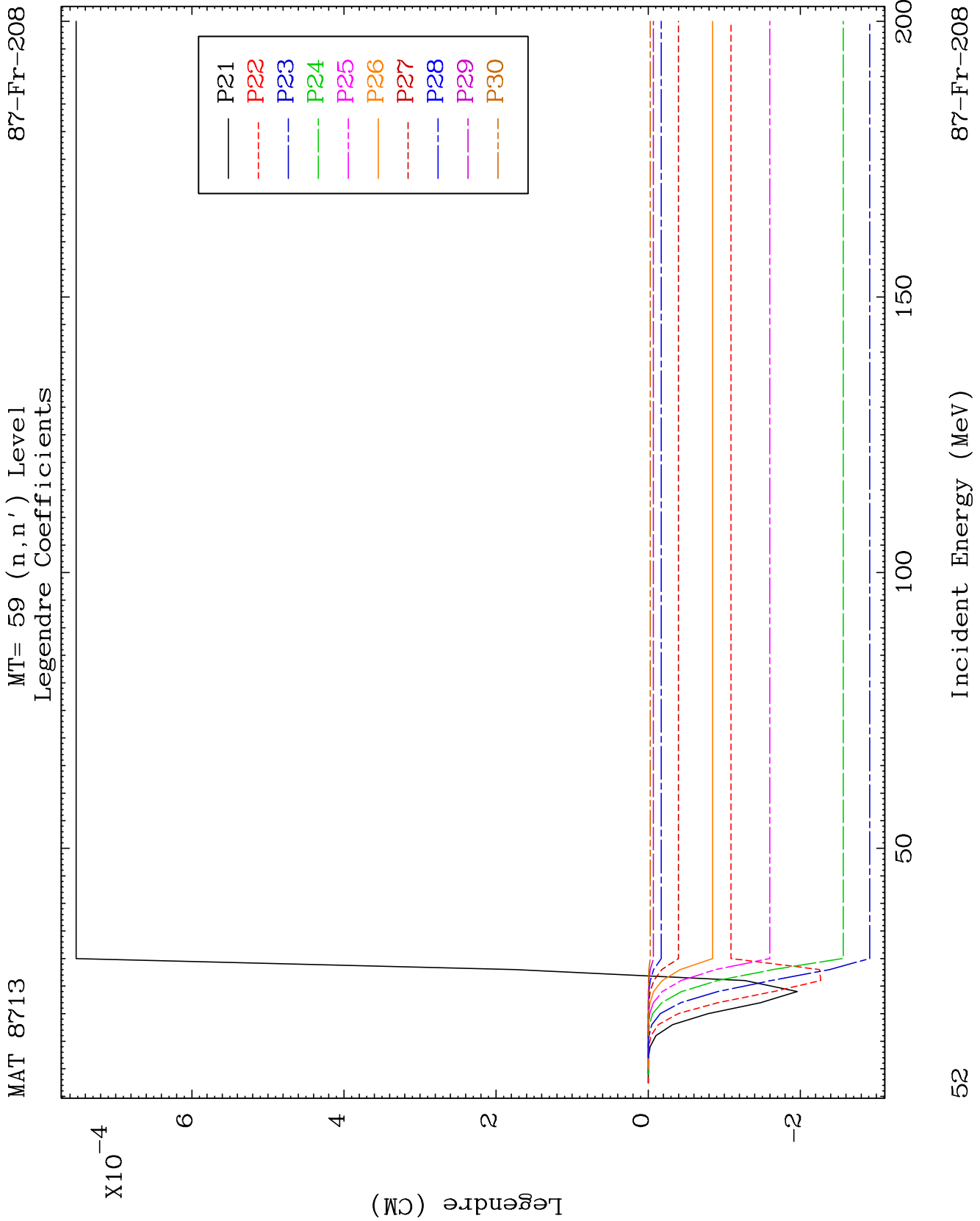
MT= 58 (n,n') Level
Legendre Coefficients

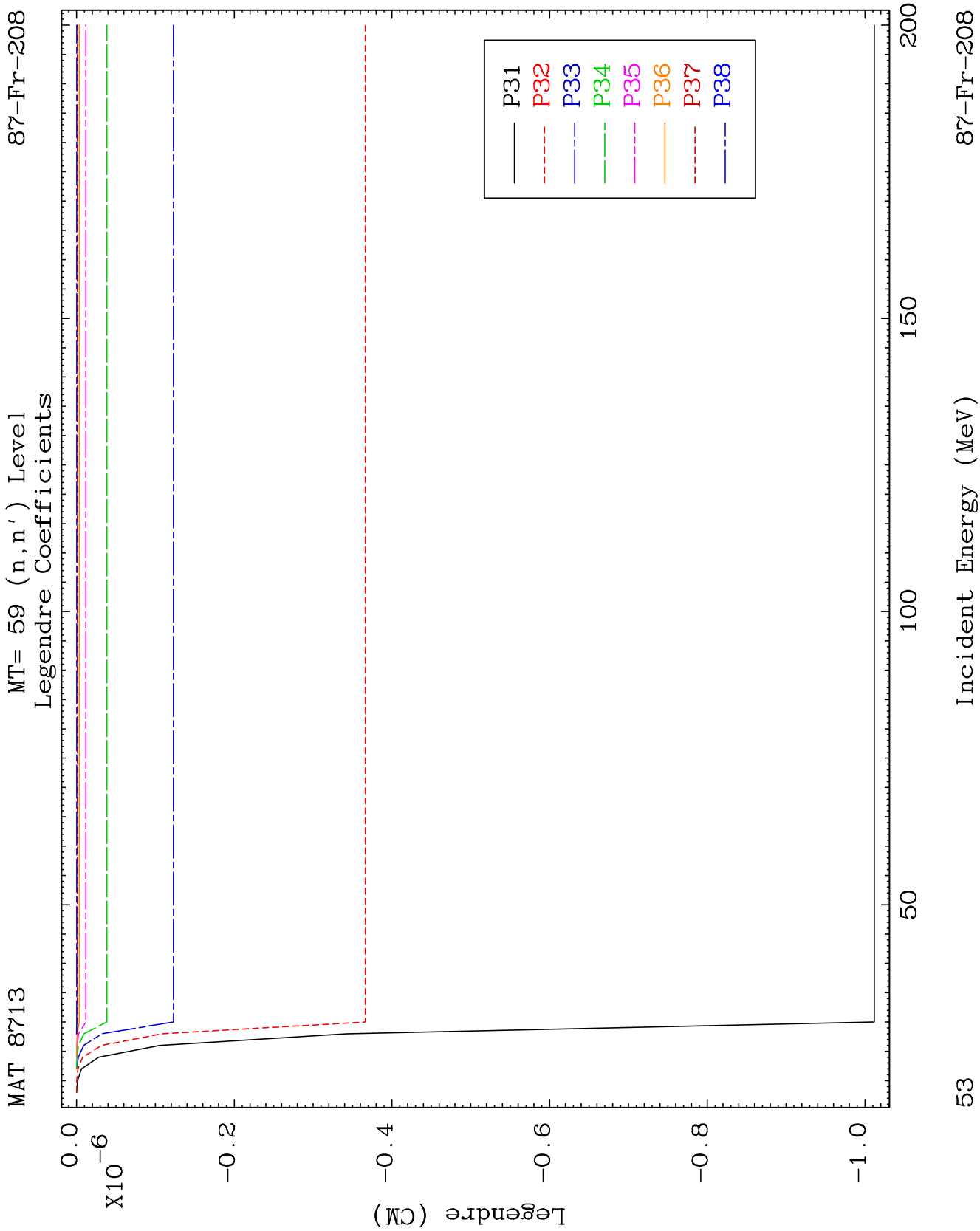
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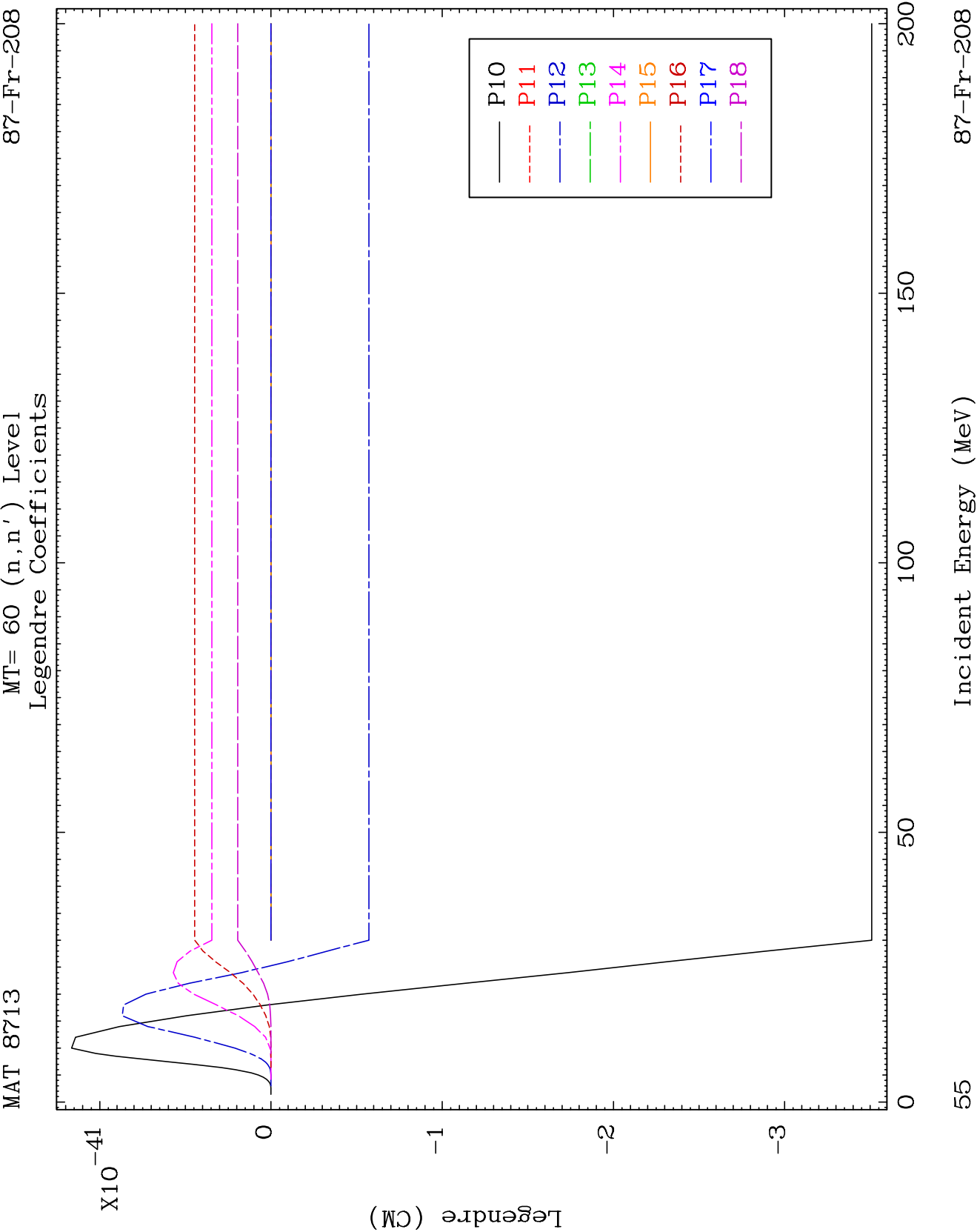


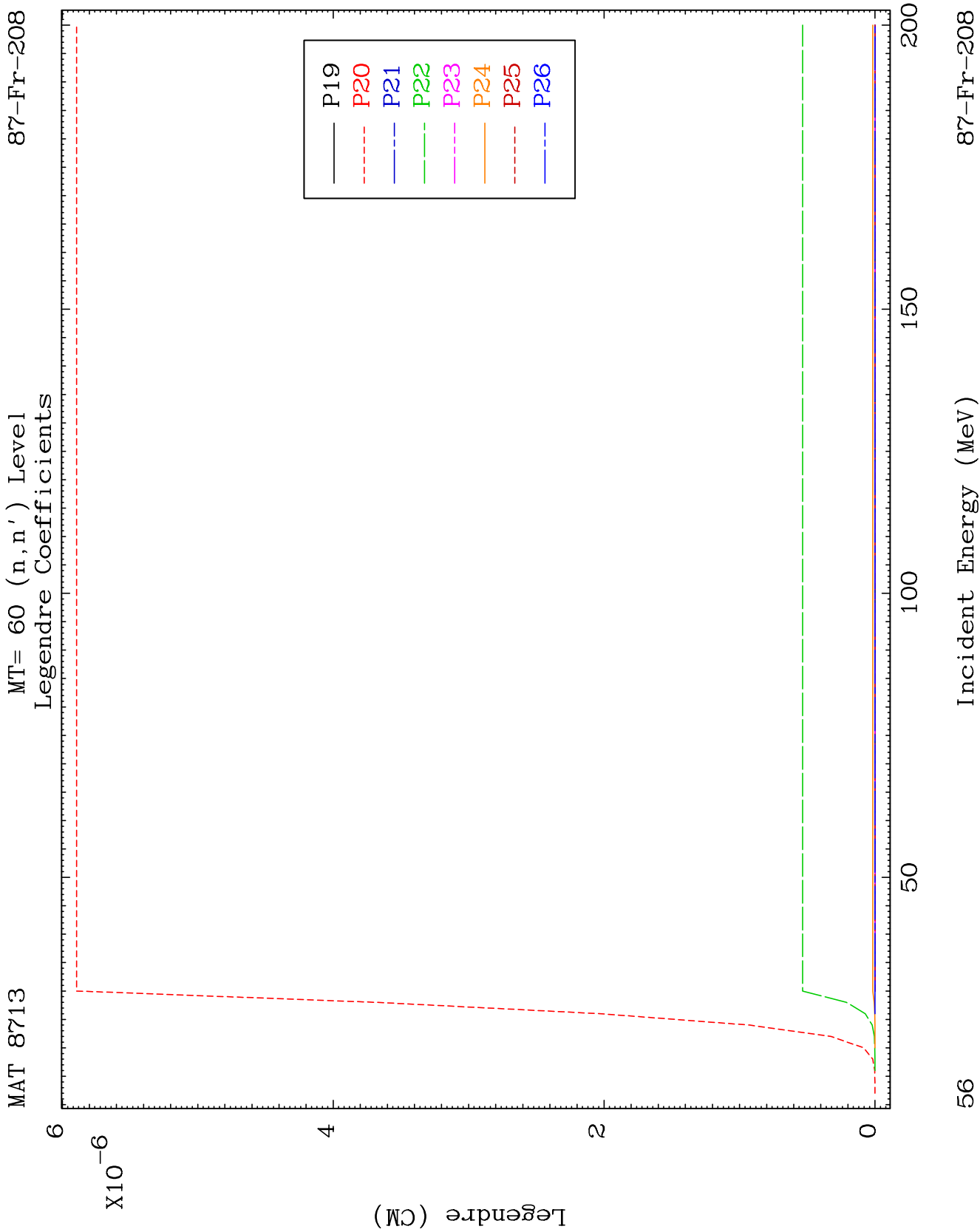








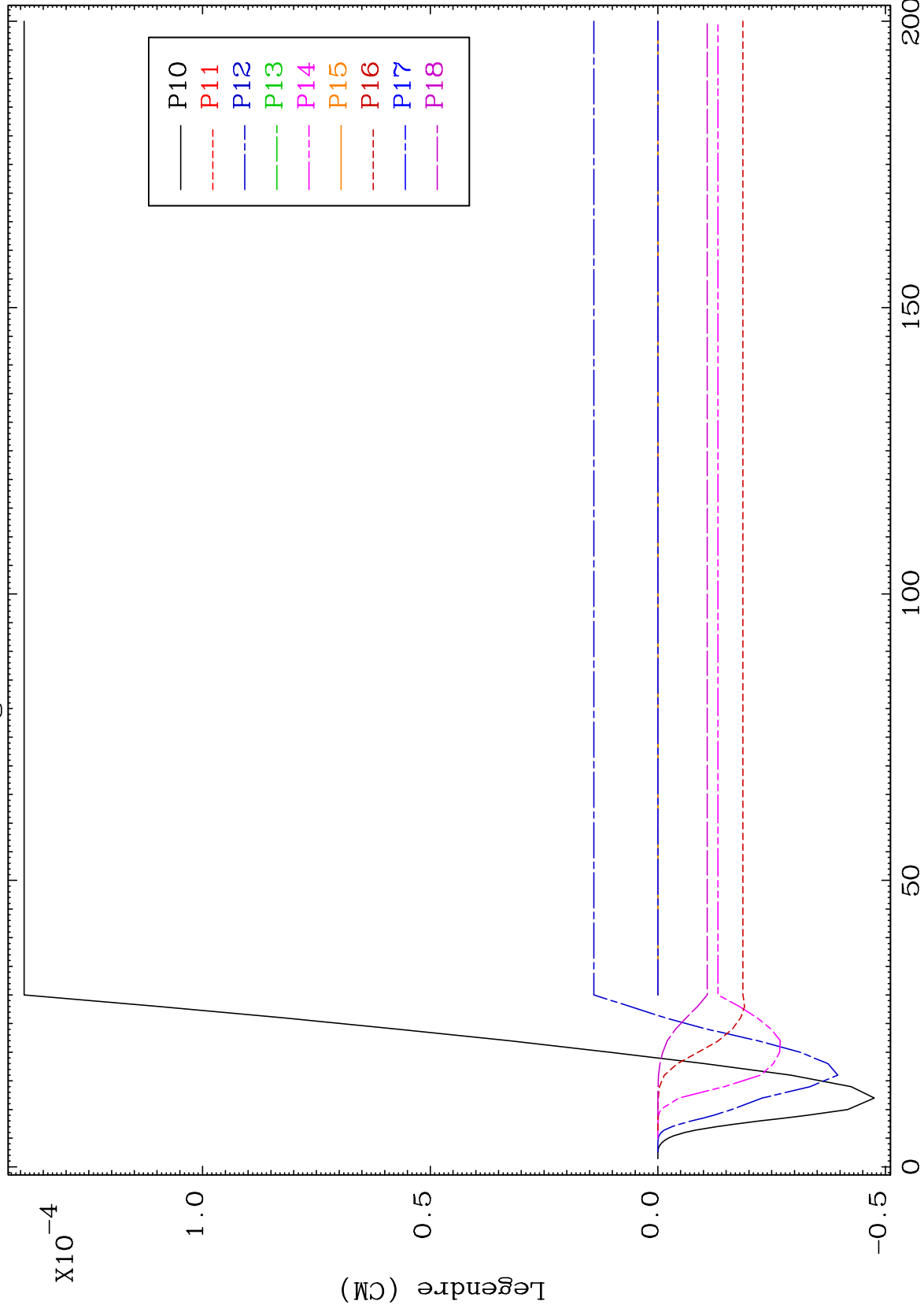




MAT 8713

MT= 61 (n,n') Level
Legendre Coefficients

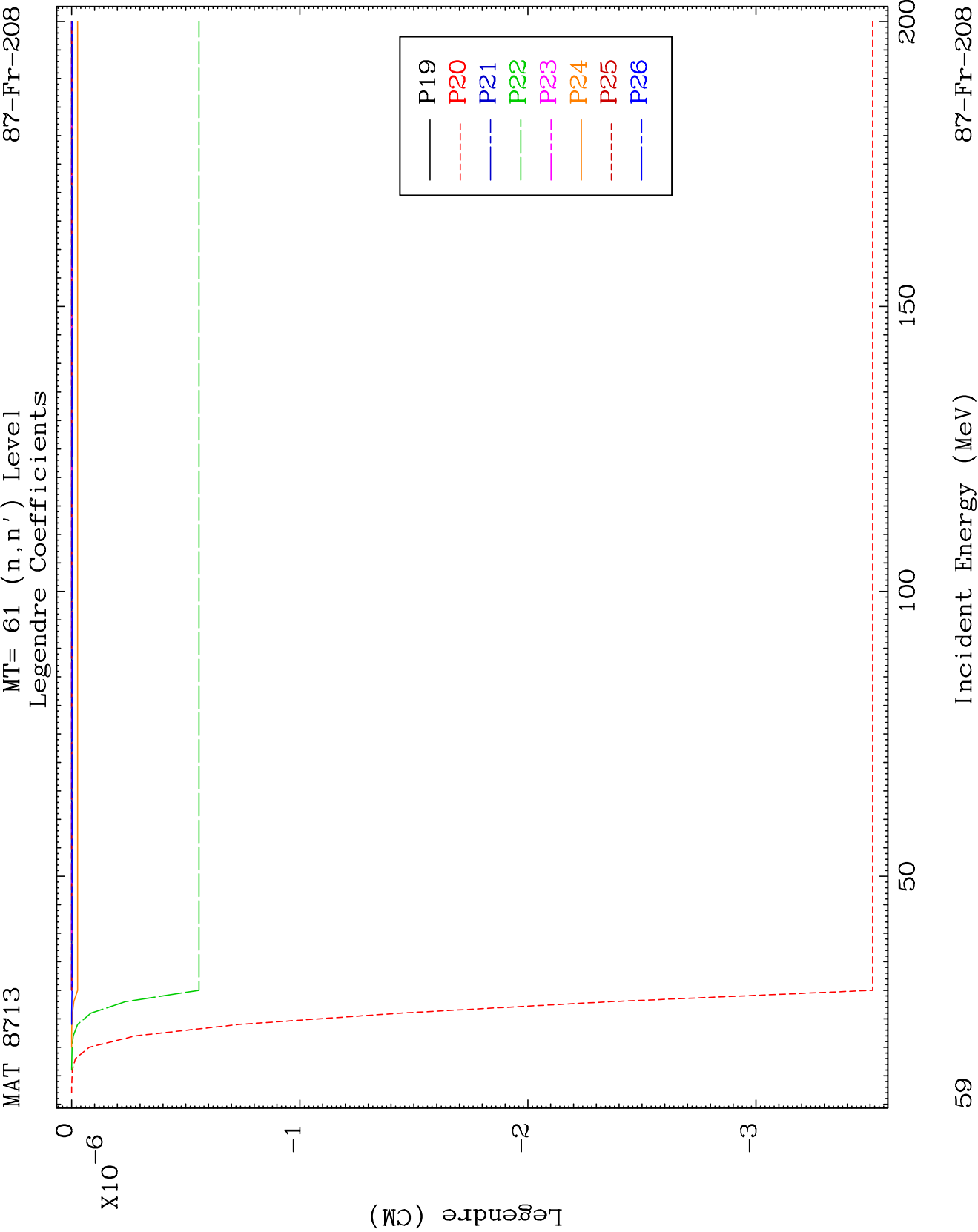
87-Fr-208



58

Incident Energy (MeV)

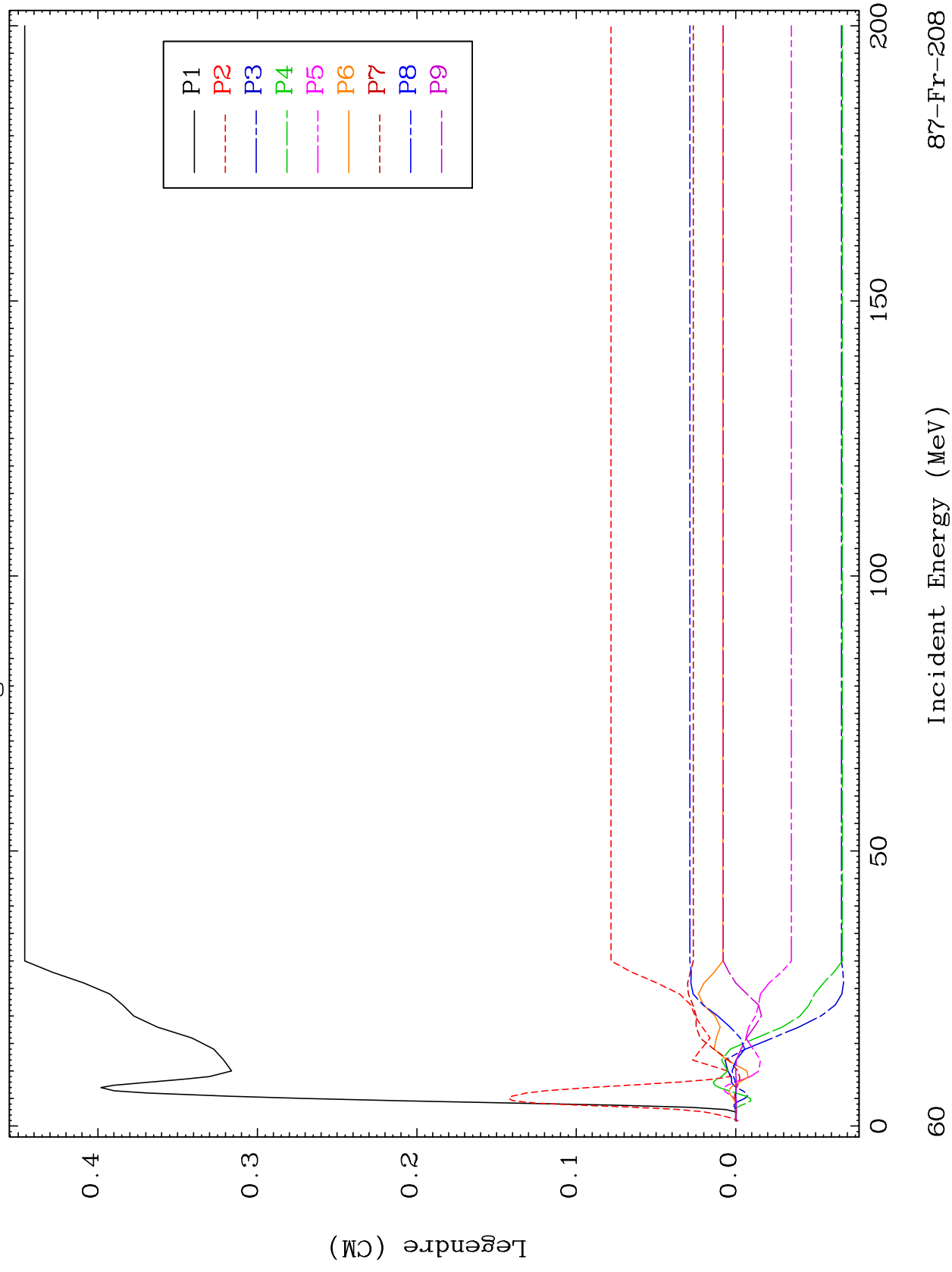
87-Fr-208



MAT 8713

MT= 62 (n, n') Level
Legendre Coefficients

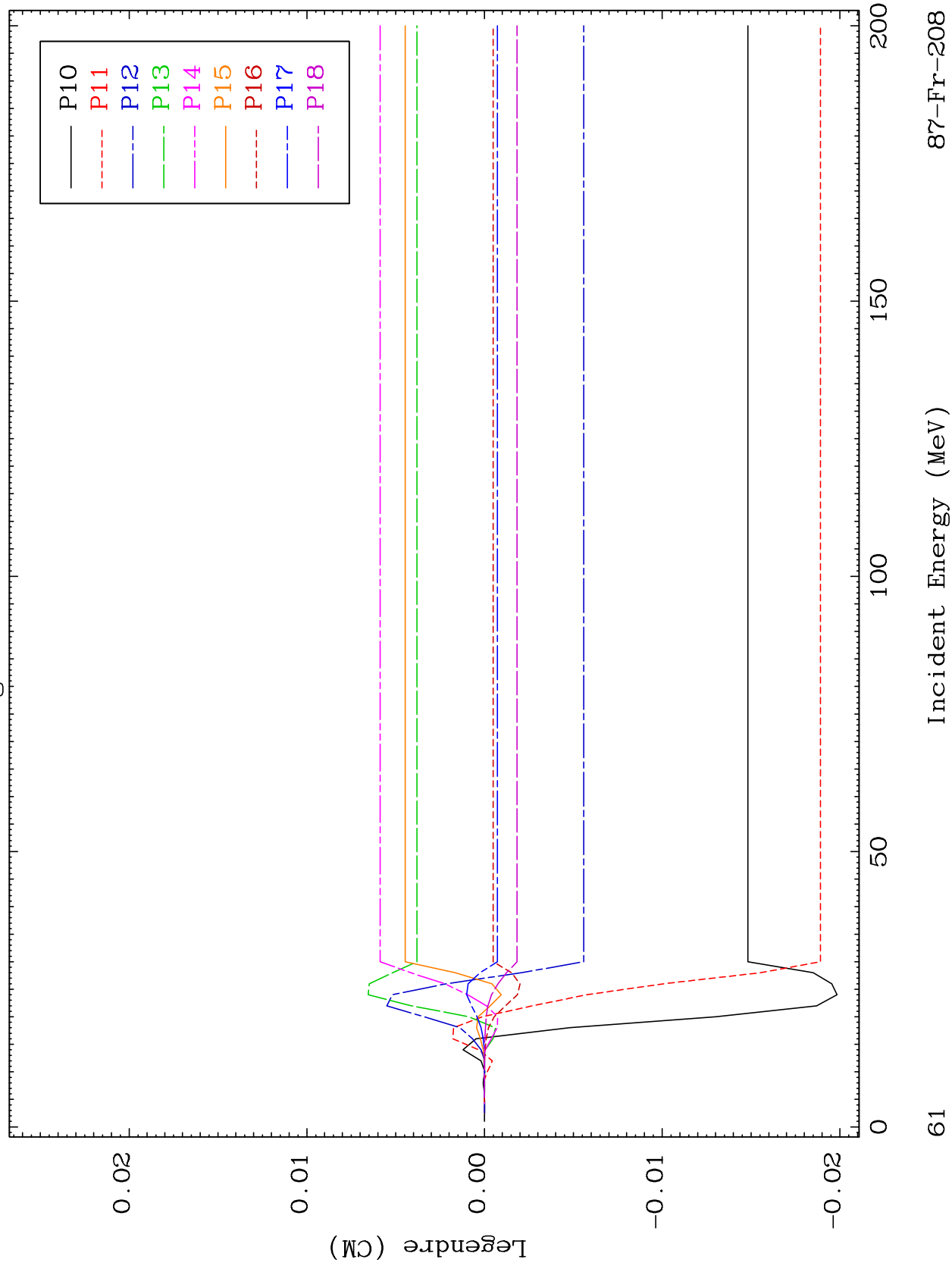
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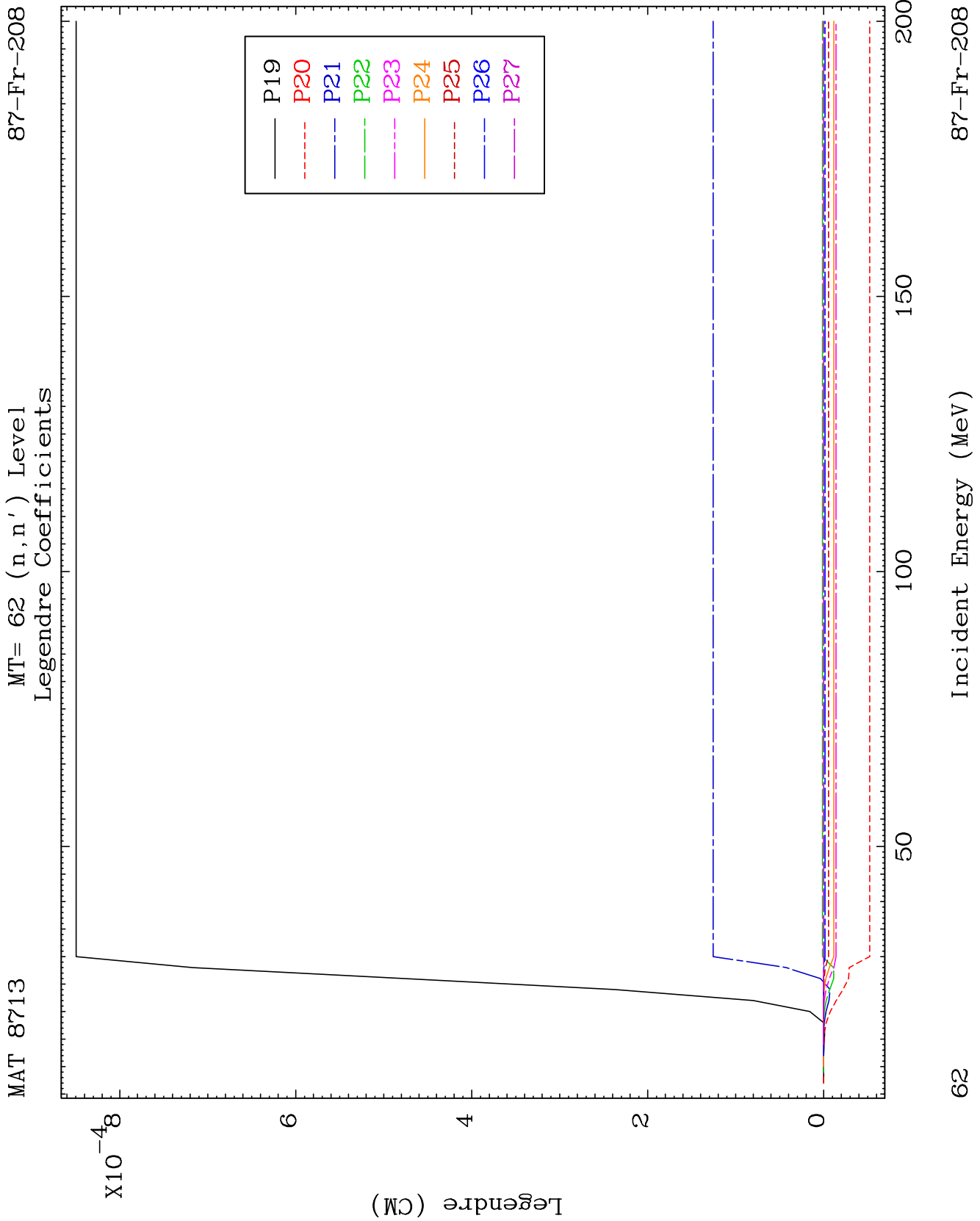


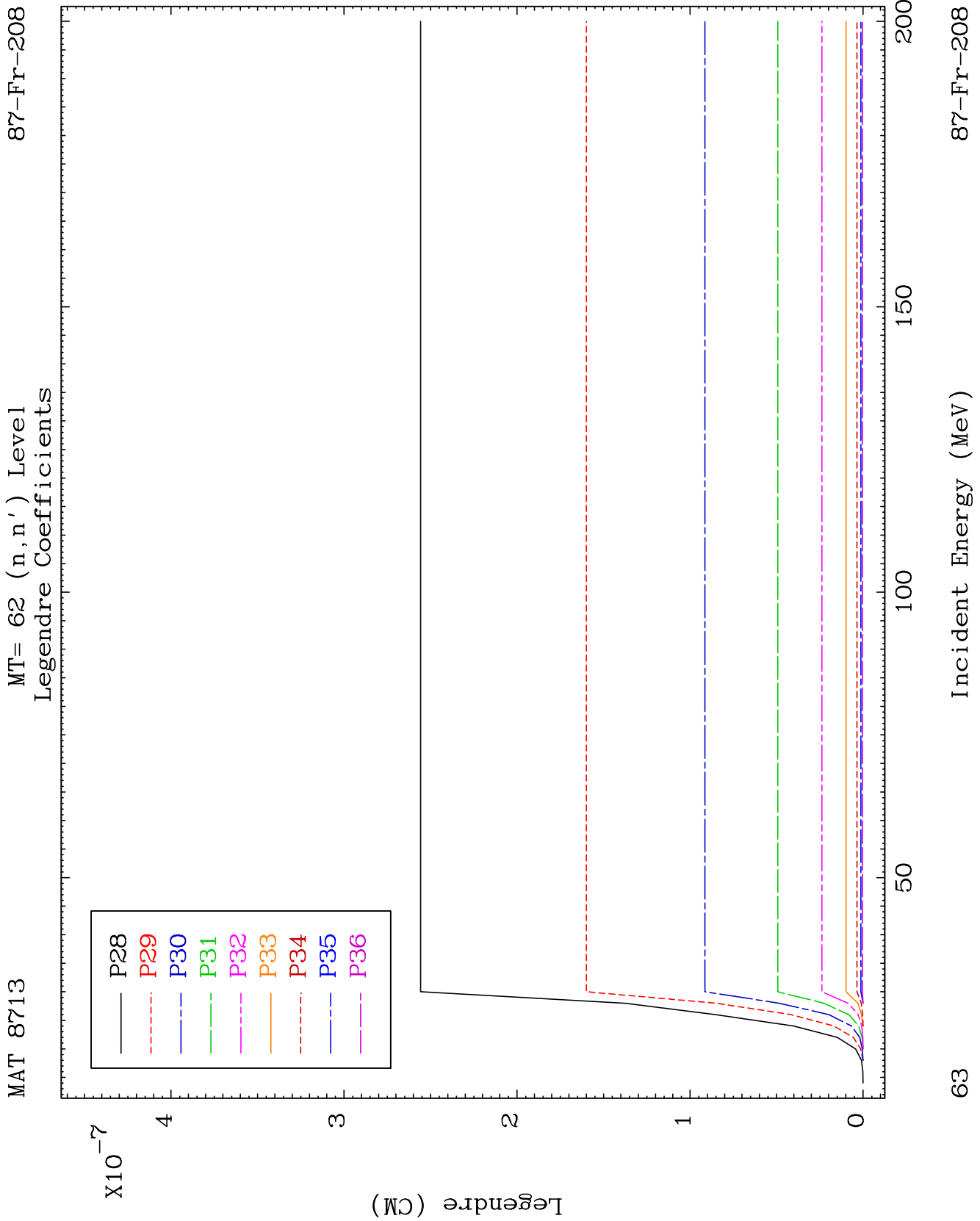
MAT 8713

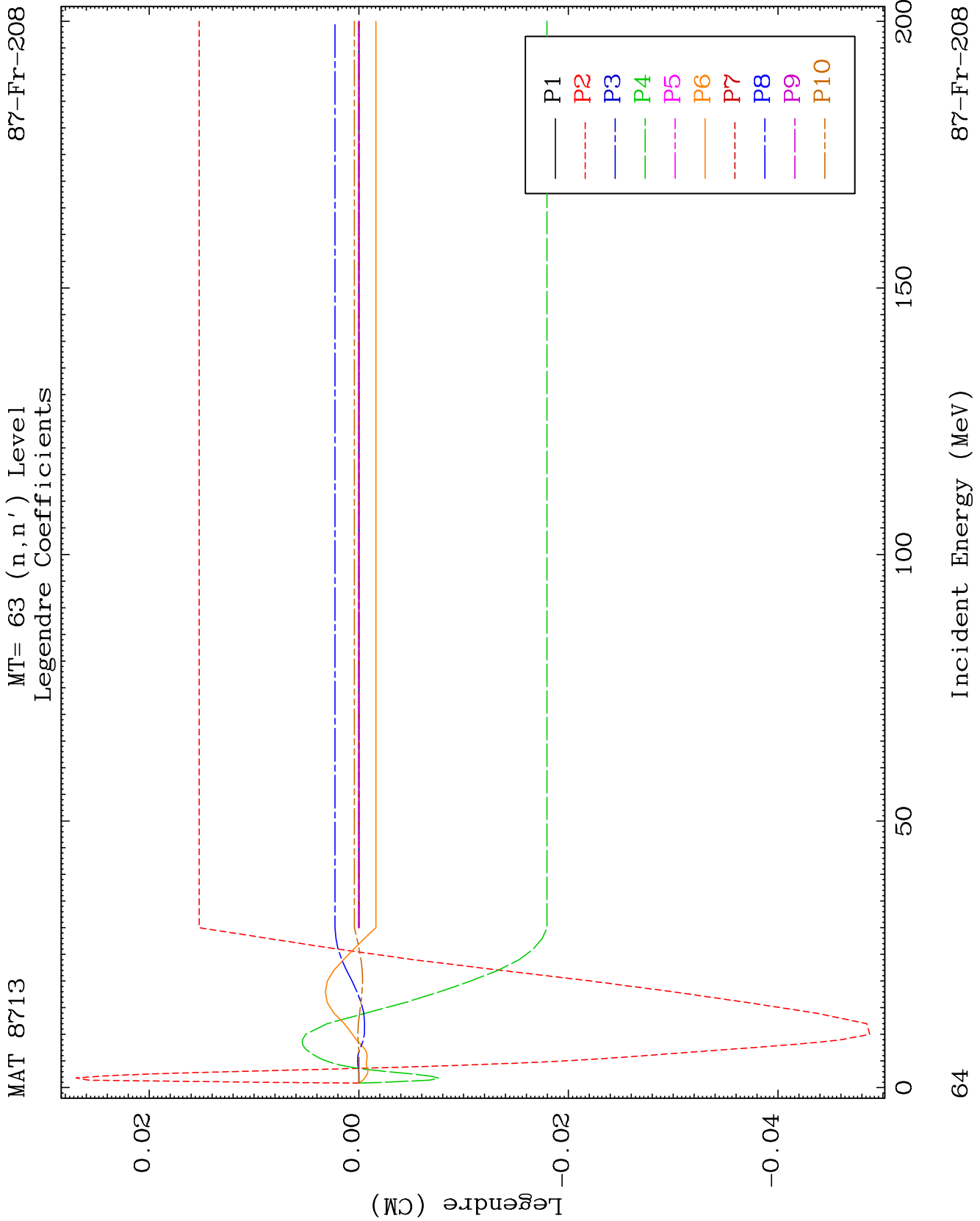
MT= 62 (n, n') Level
Legendre Coefficients

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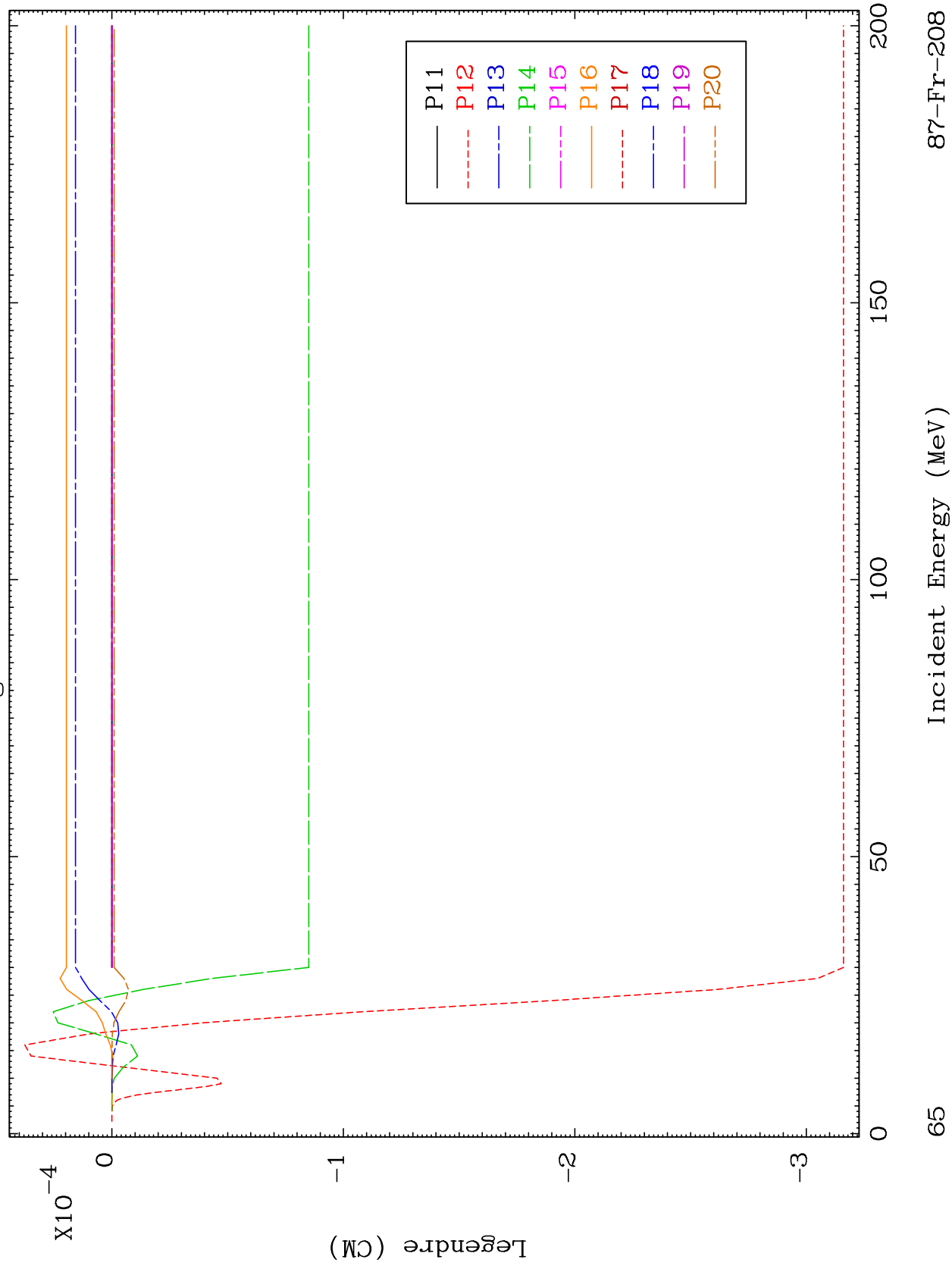


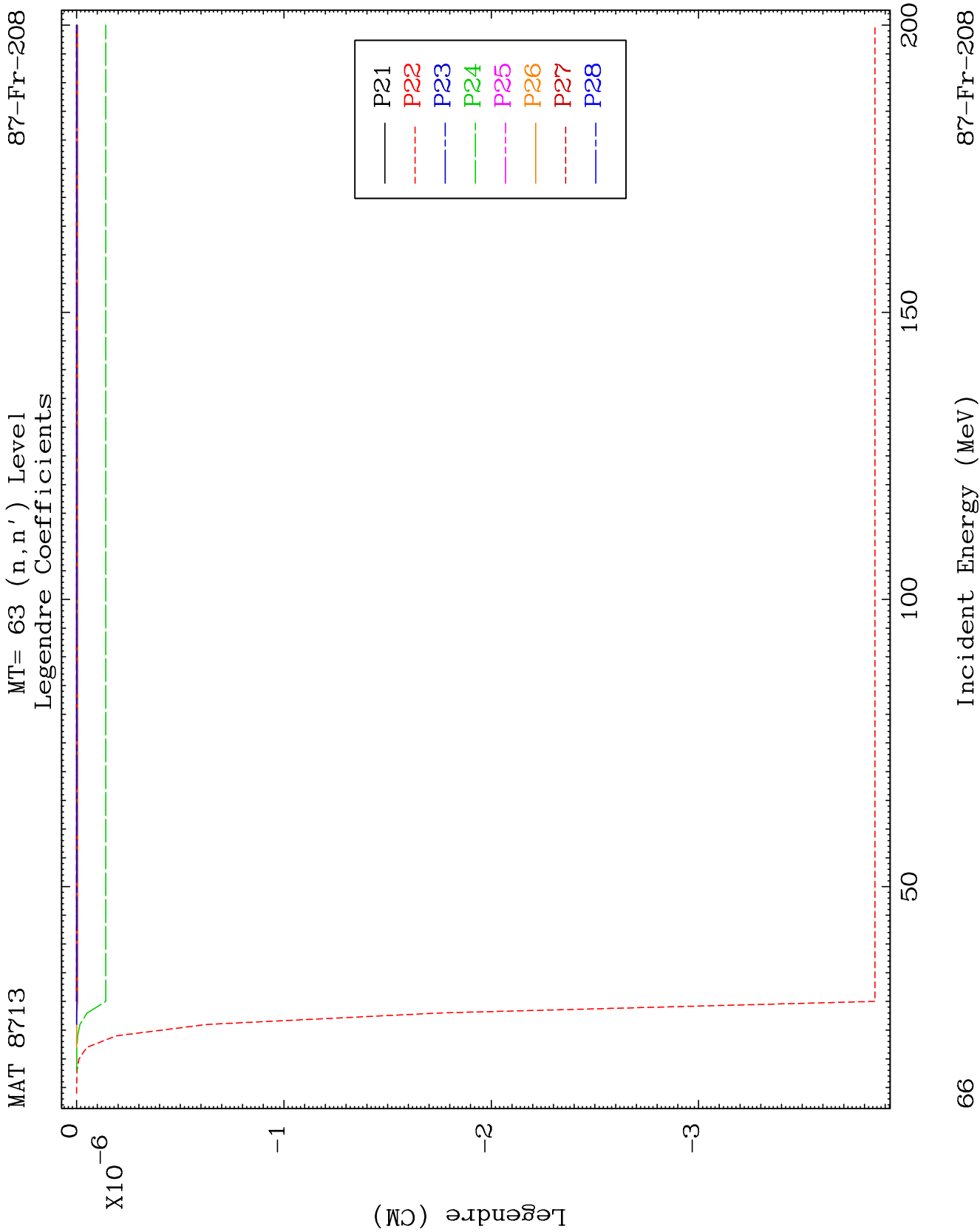


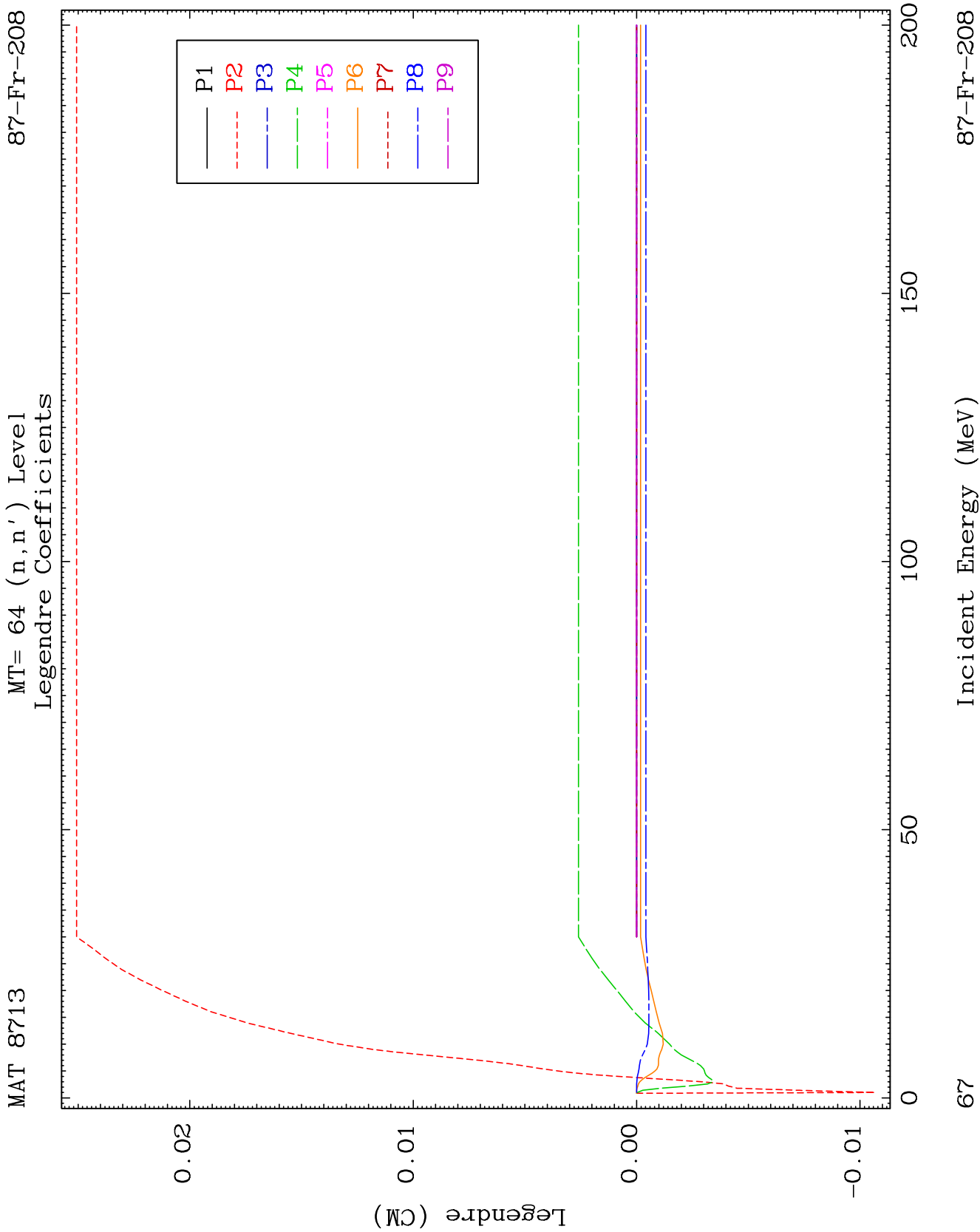
MAT 8713

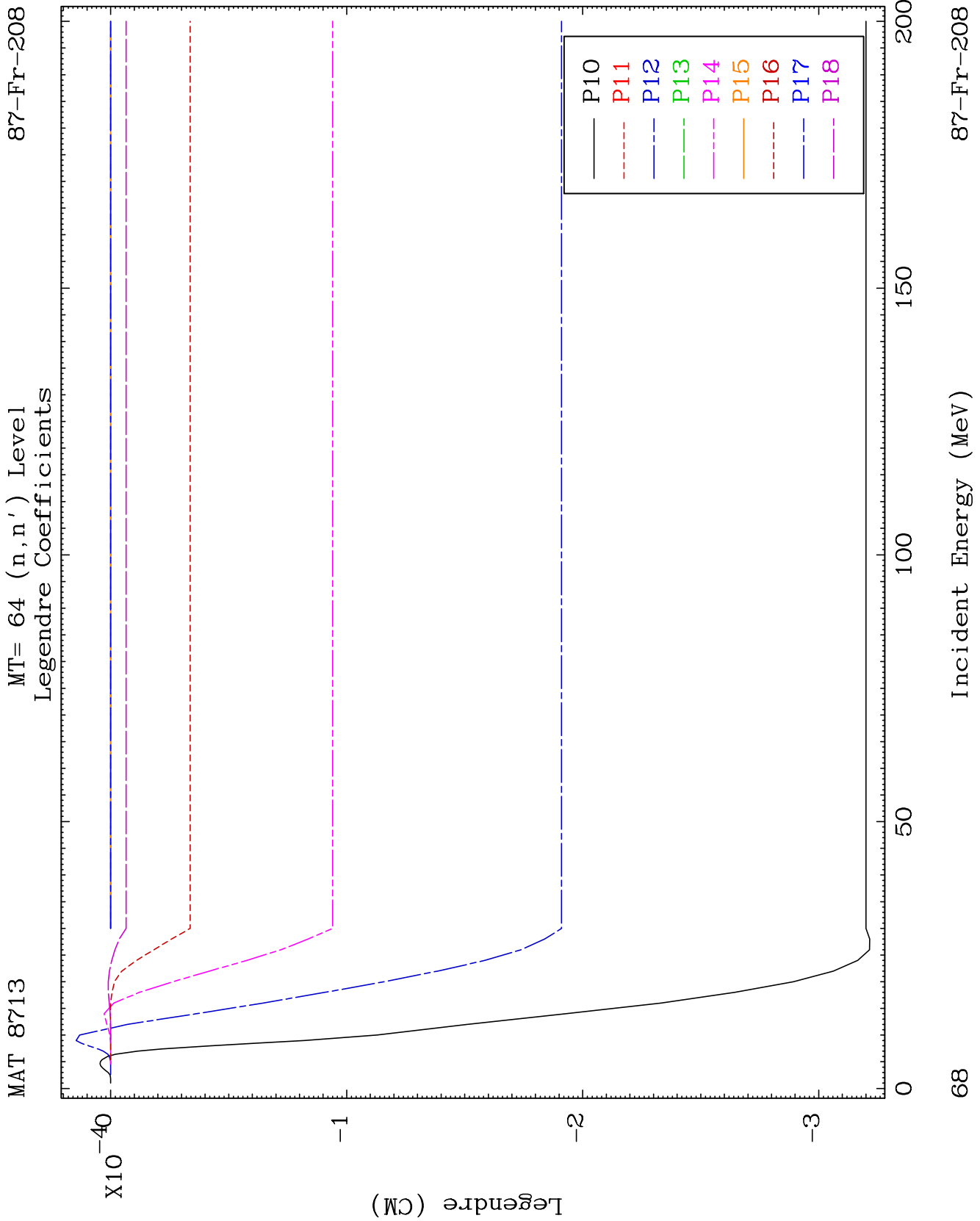
MT= 63 (n, n') Level
Legendre Coefficients

87-Fr-208





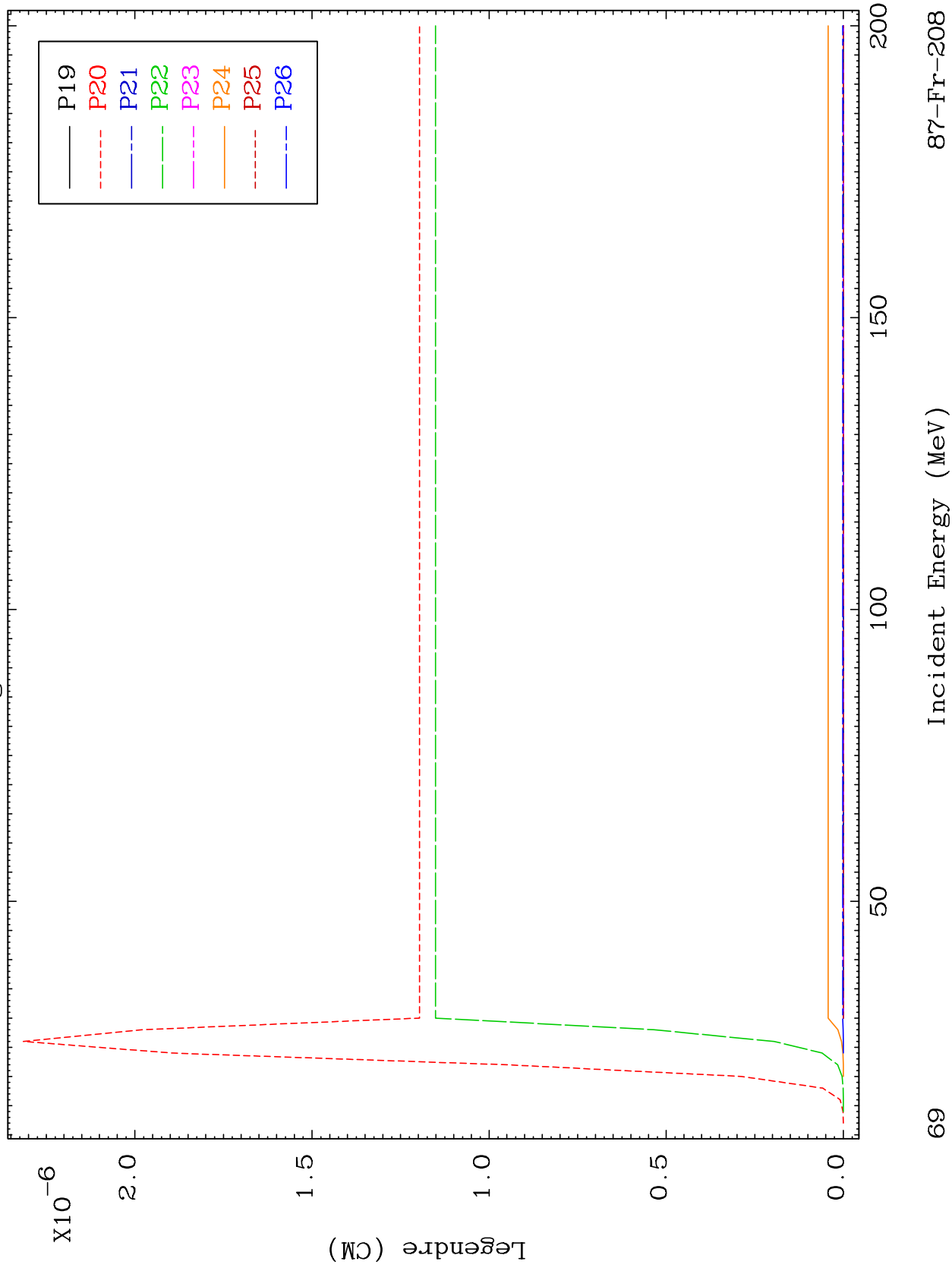


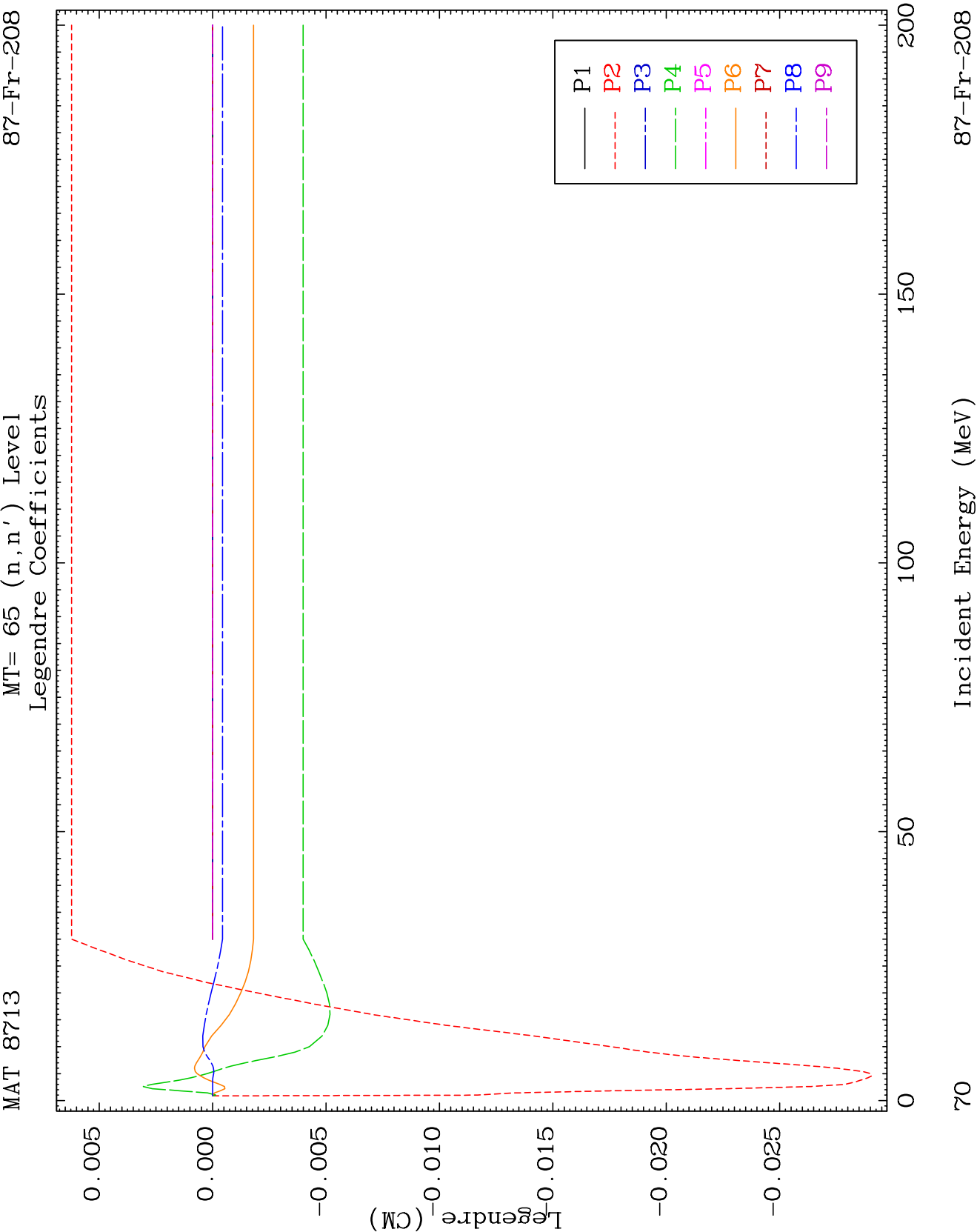


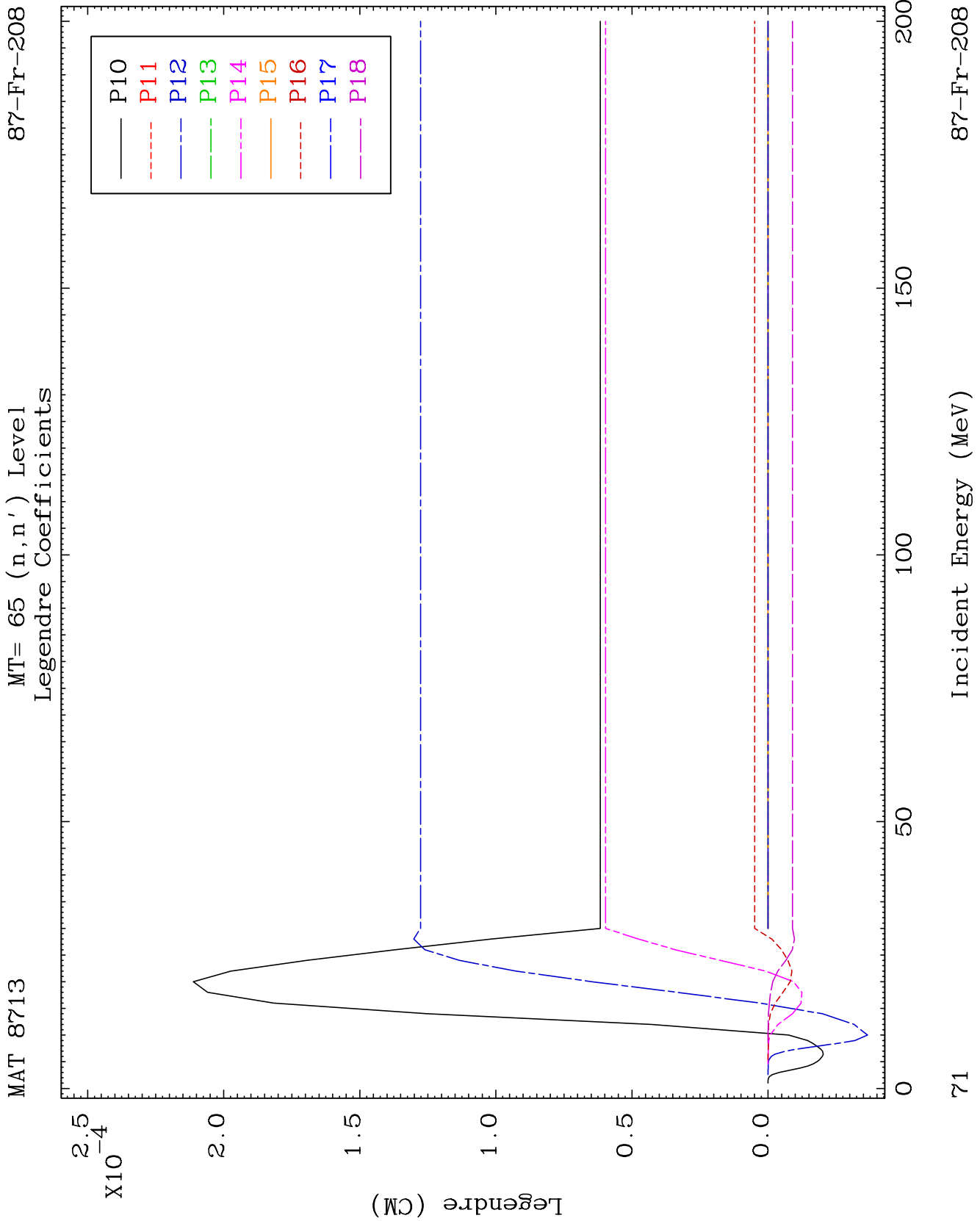
MAT 8713

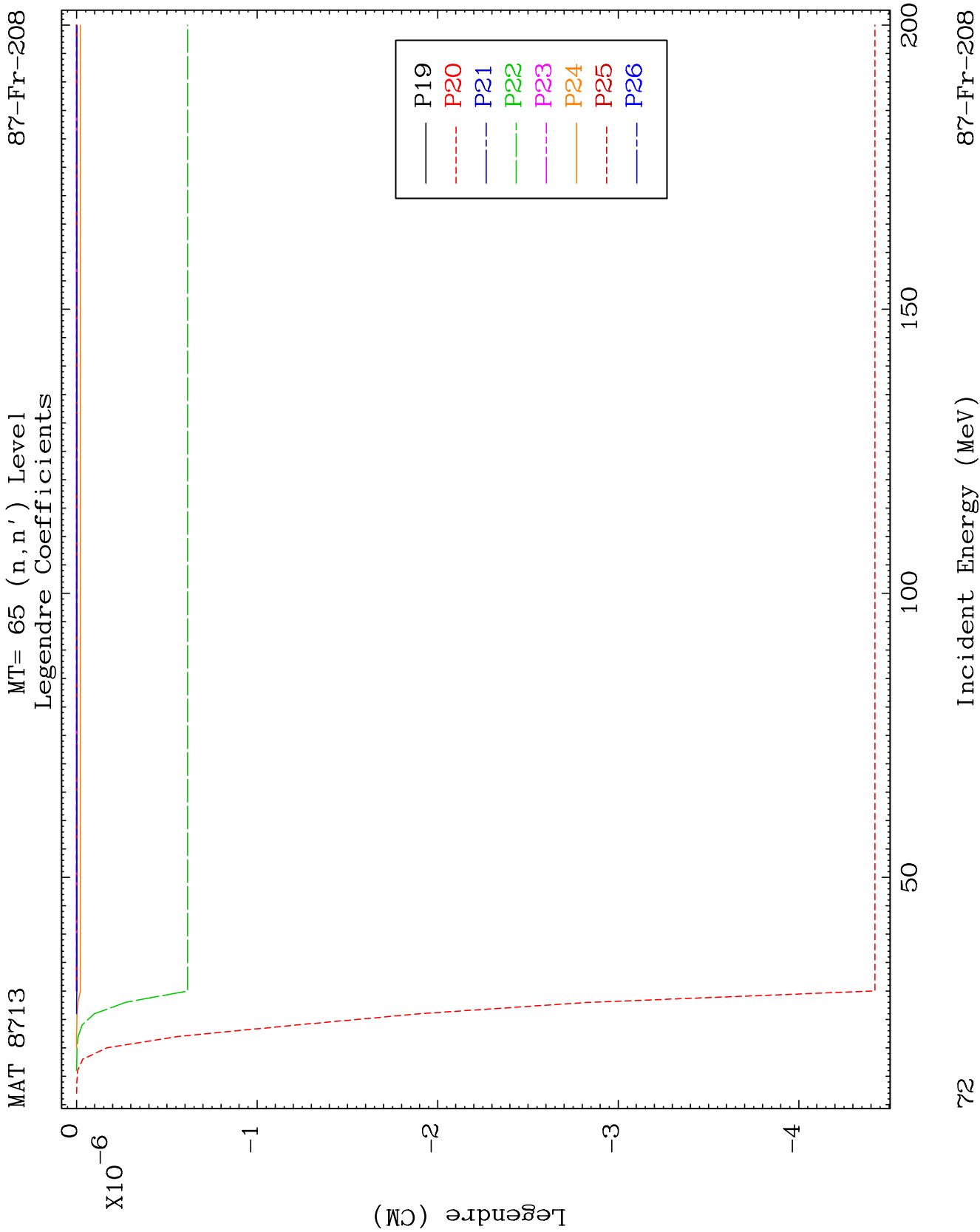
MT= 64 (n, n') Level
Legendre Coefficients

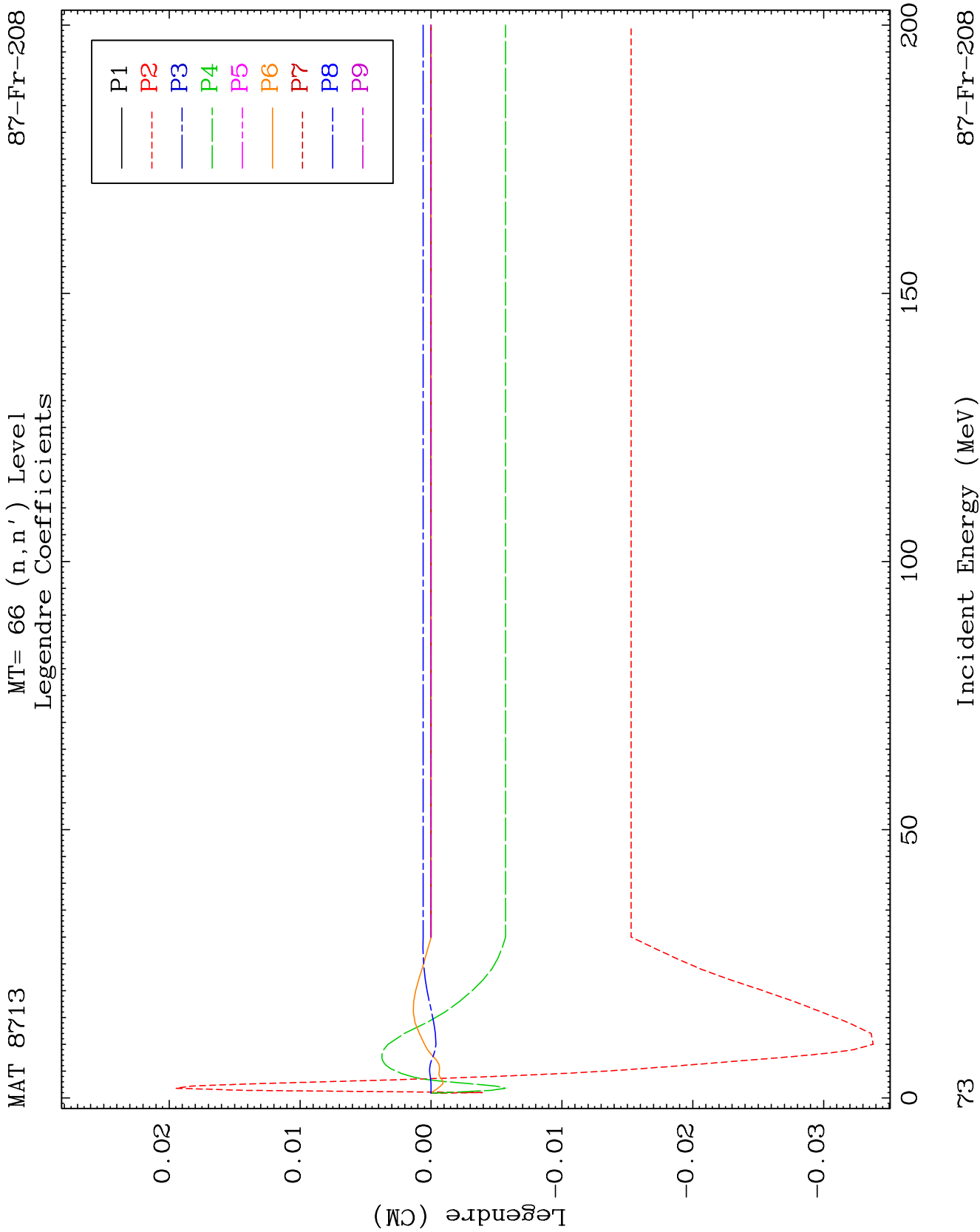
87-Fr-208

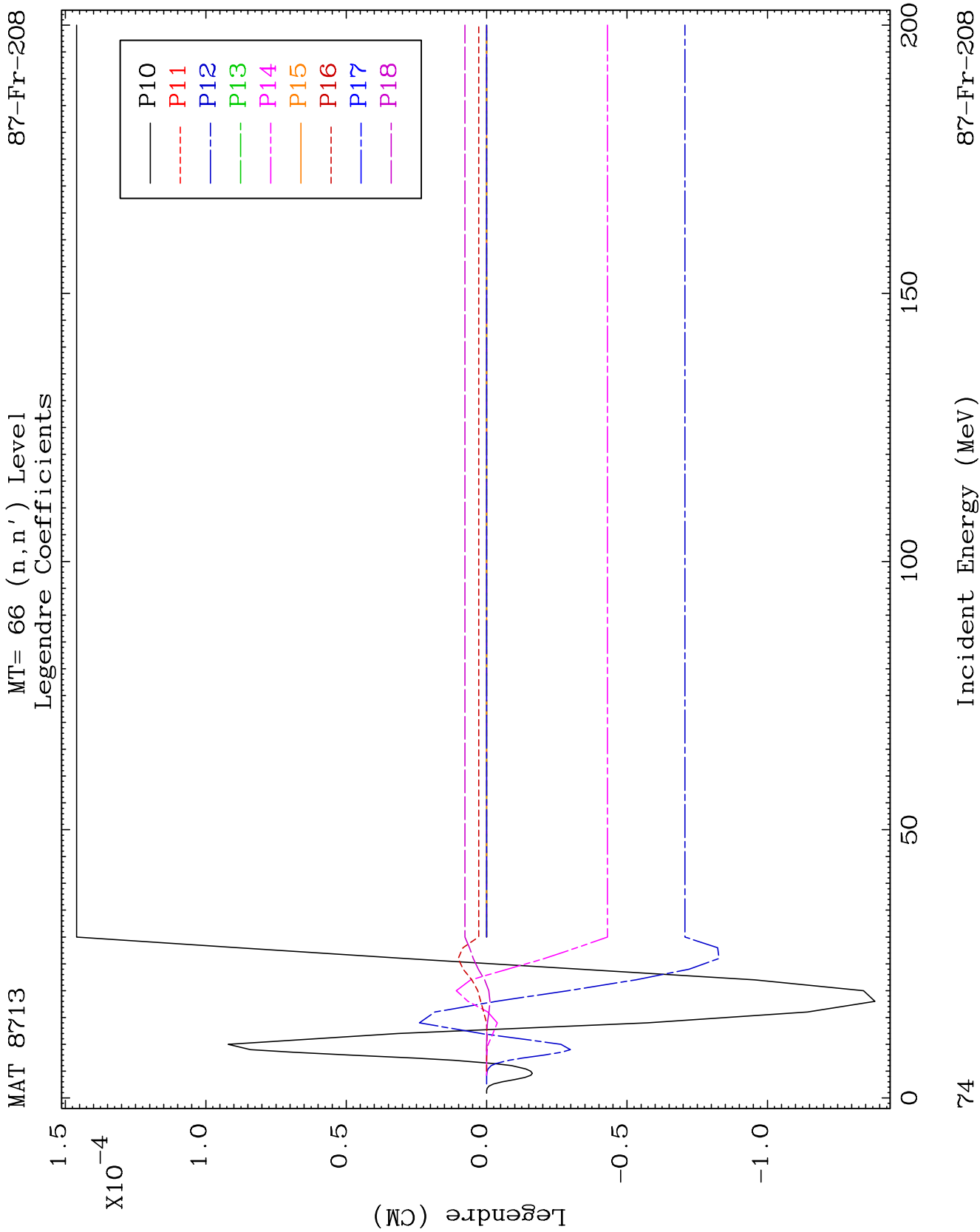


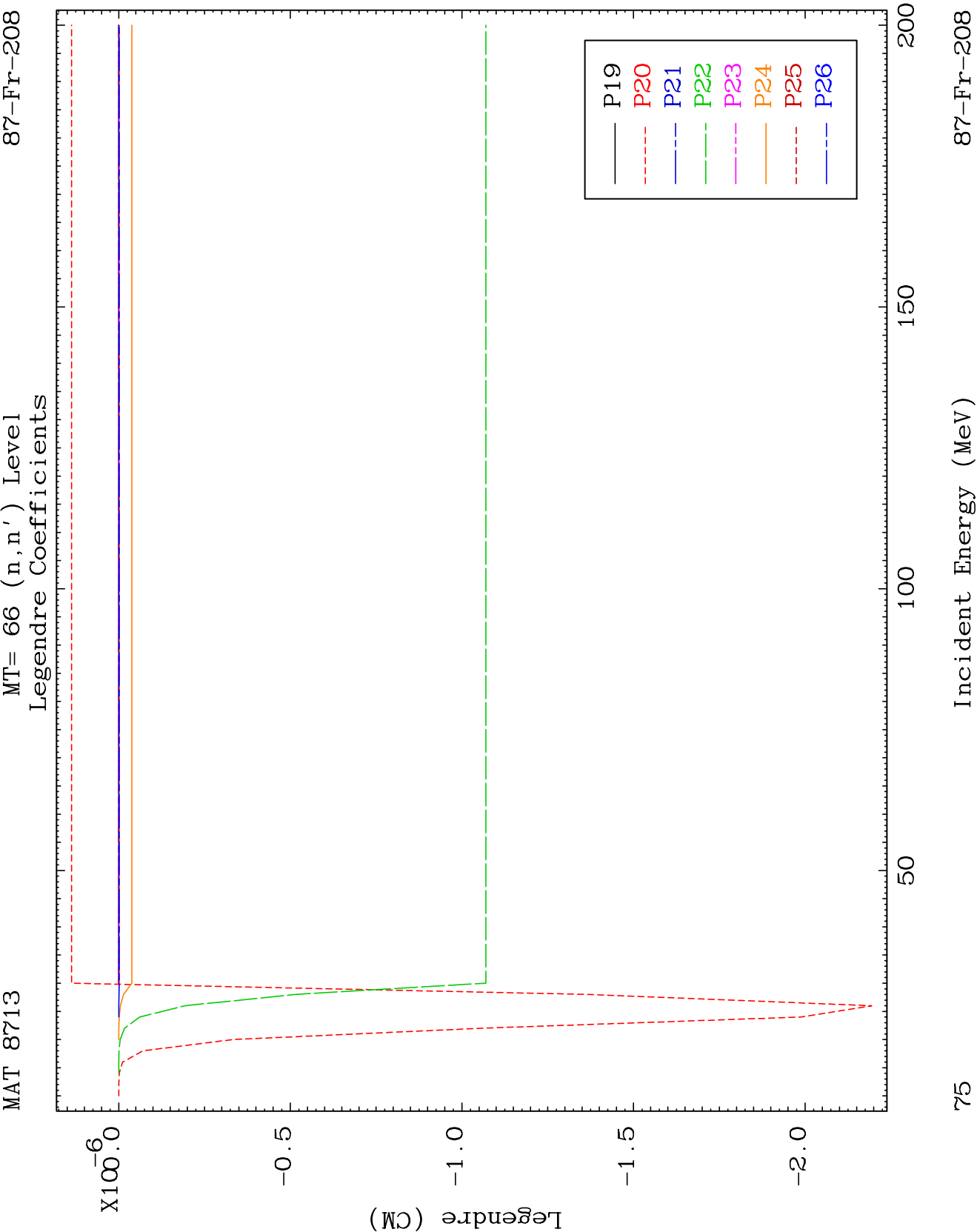


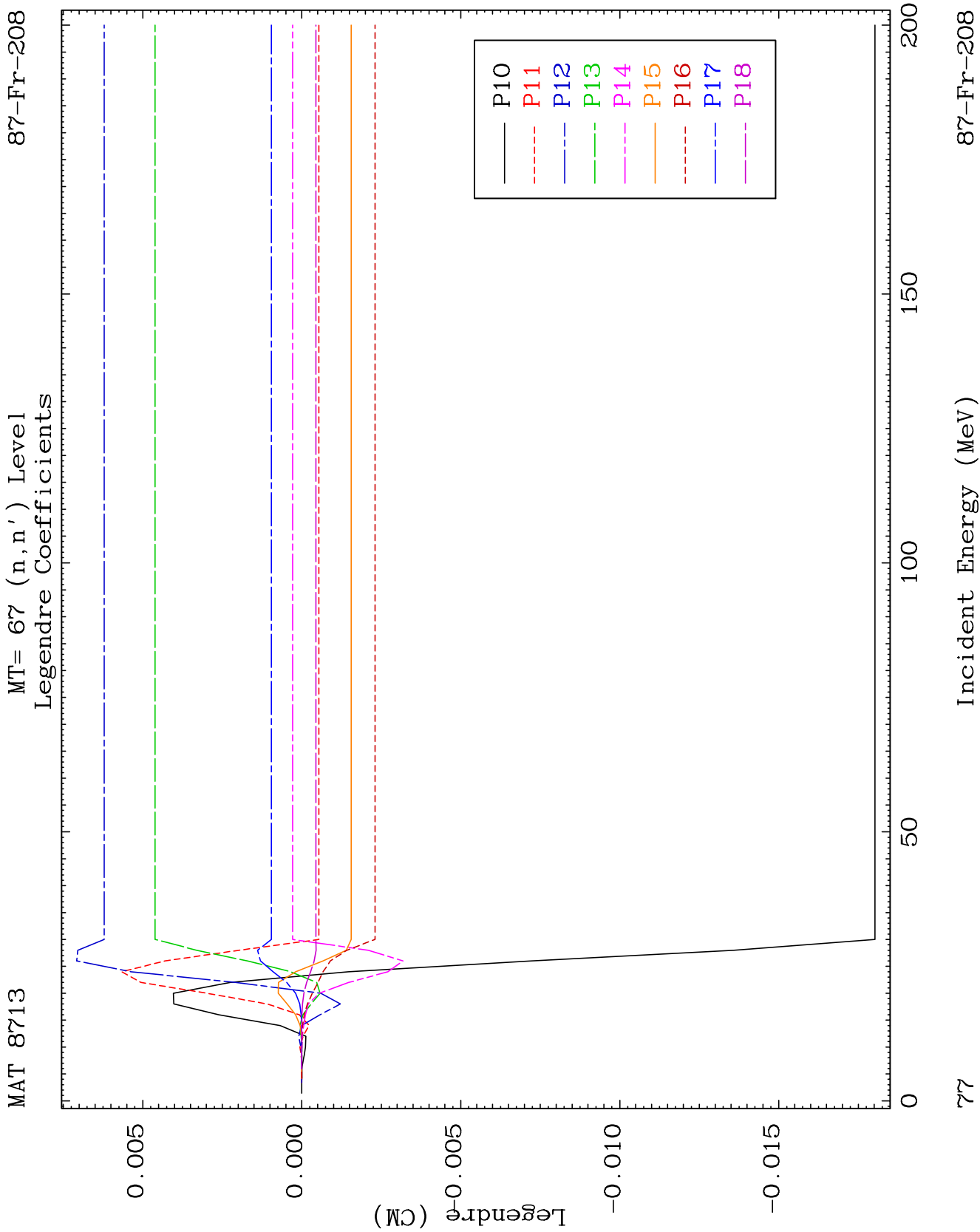


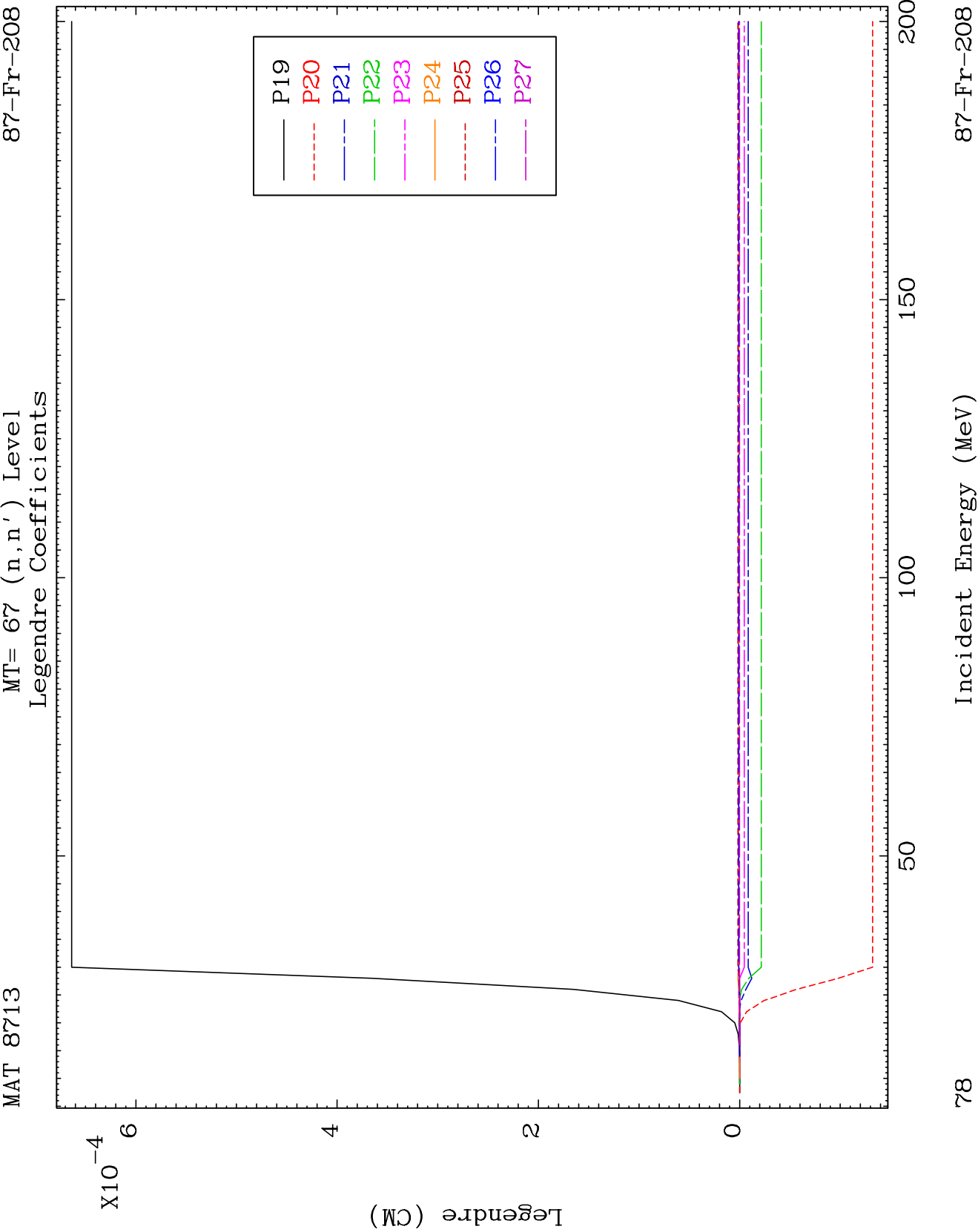


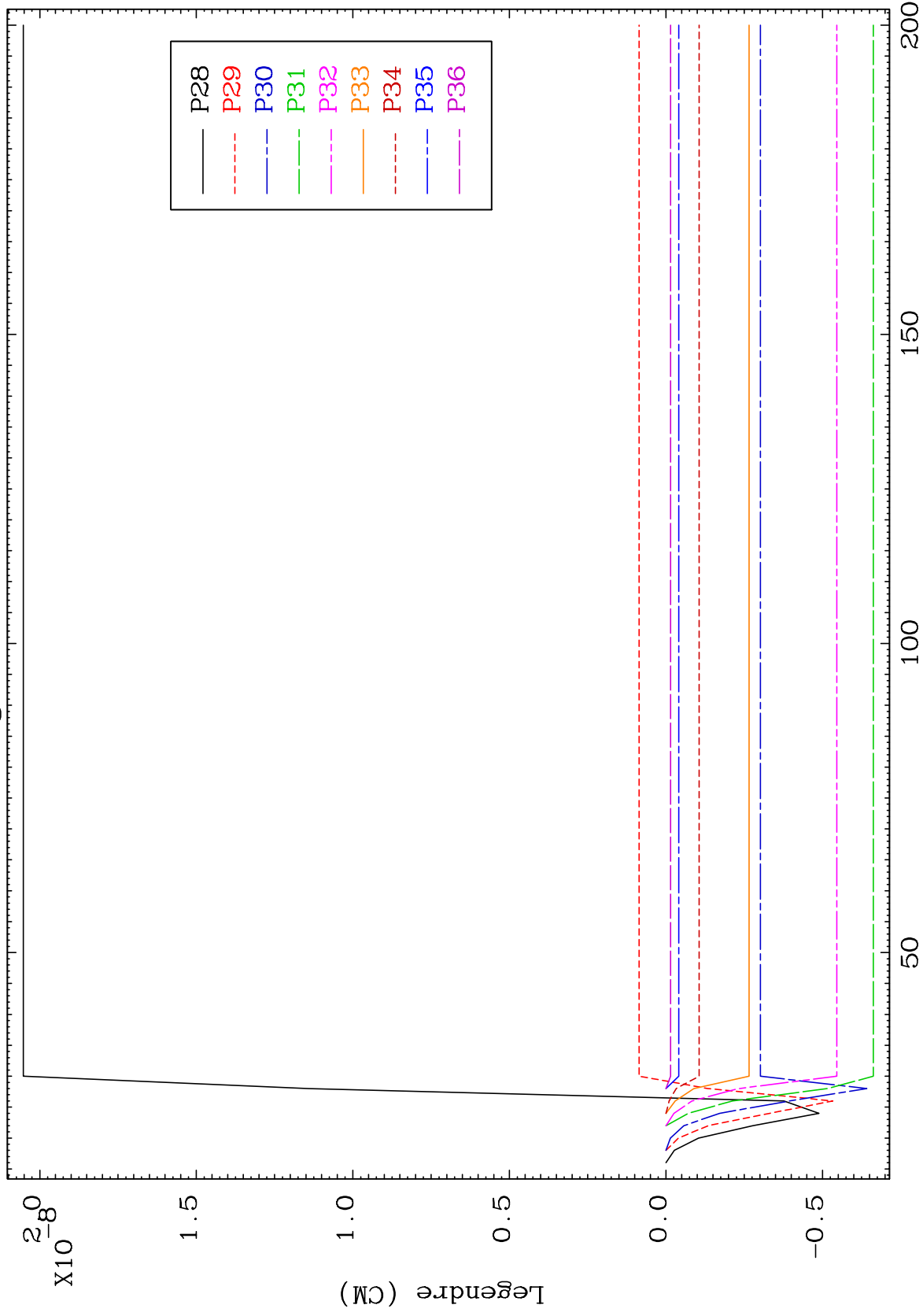


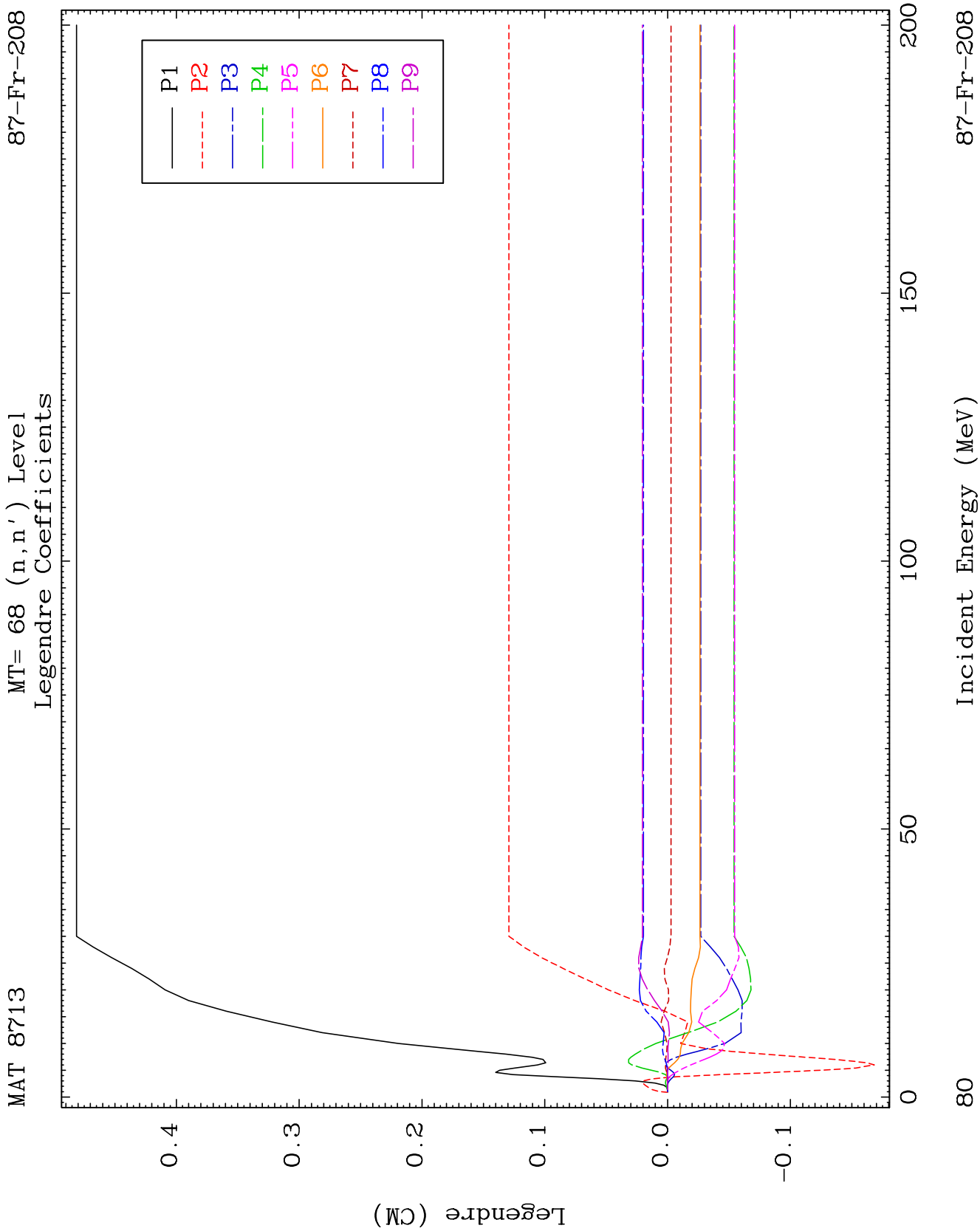


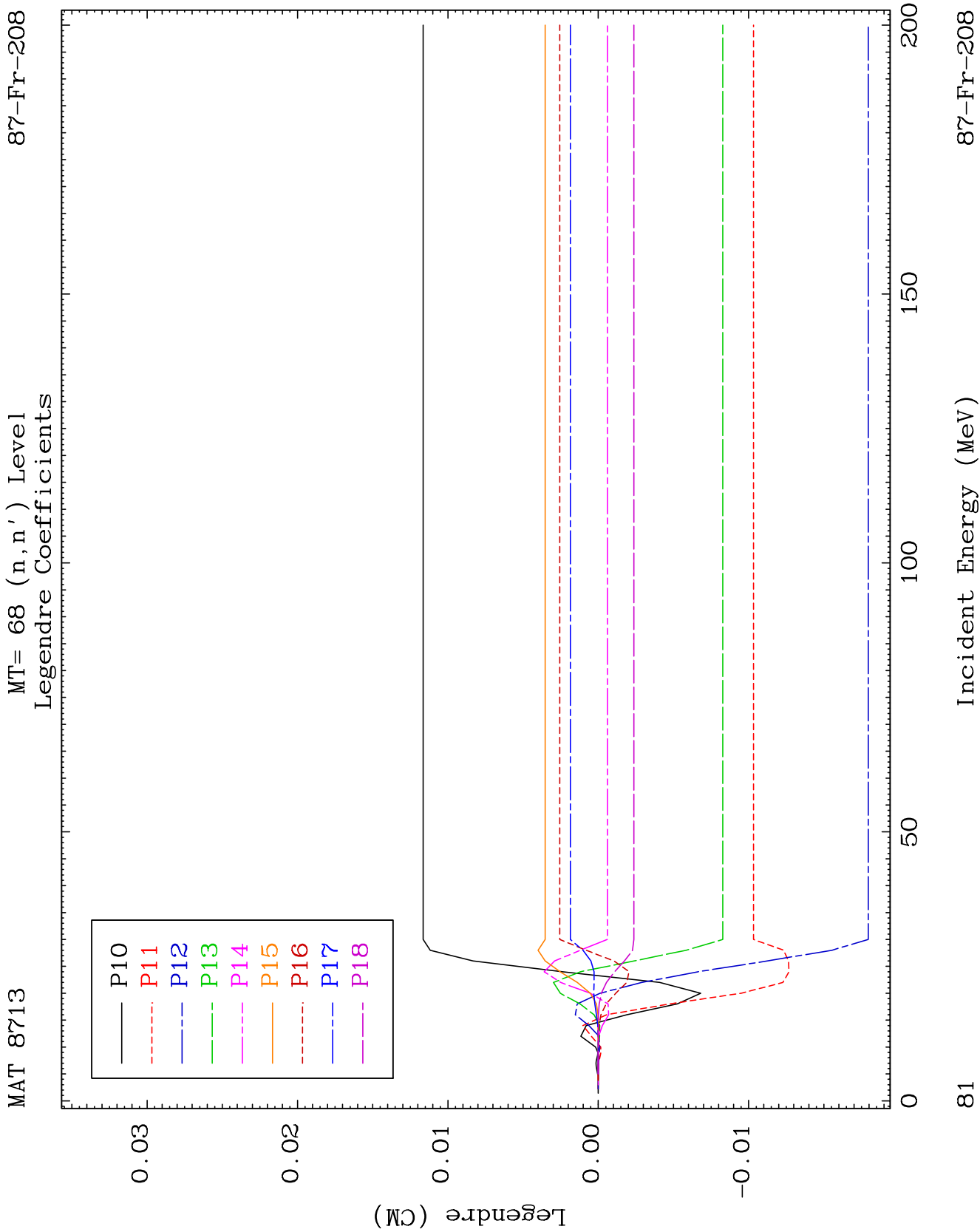


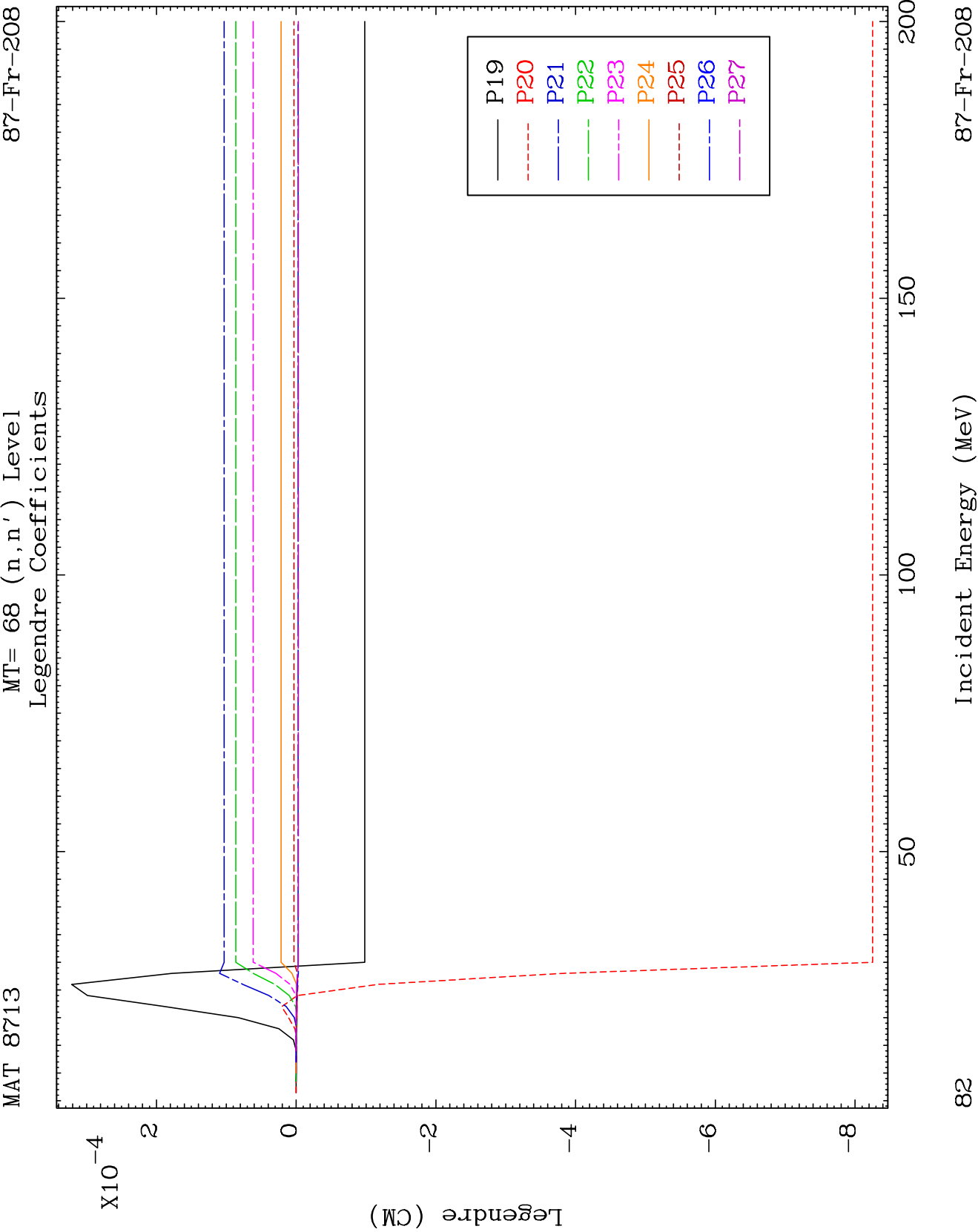


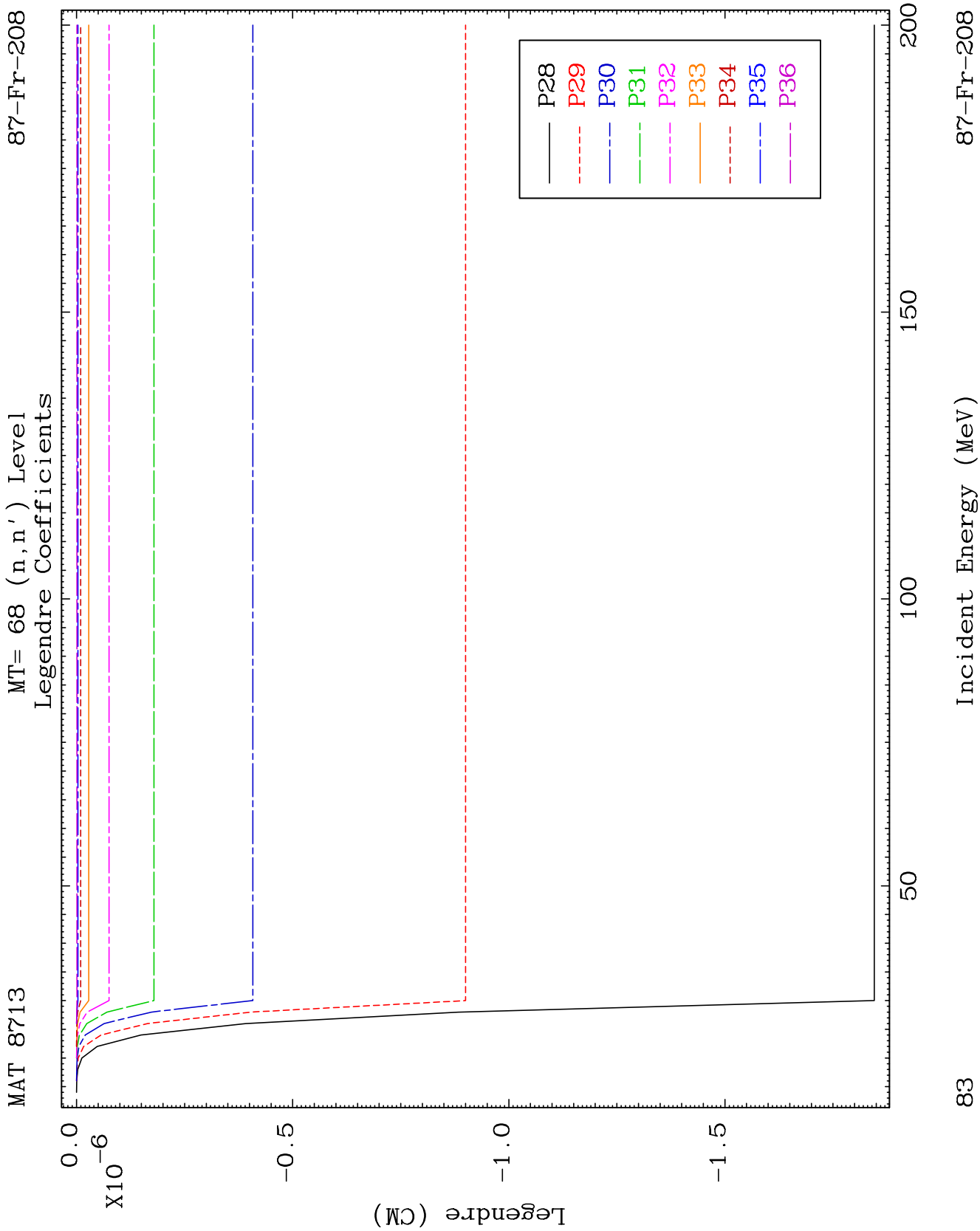








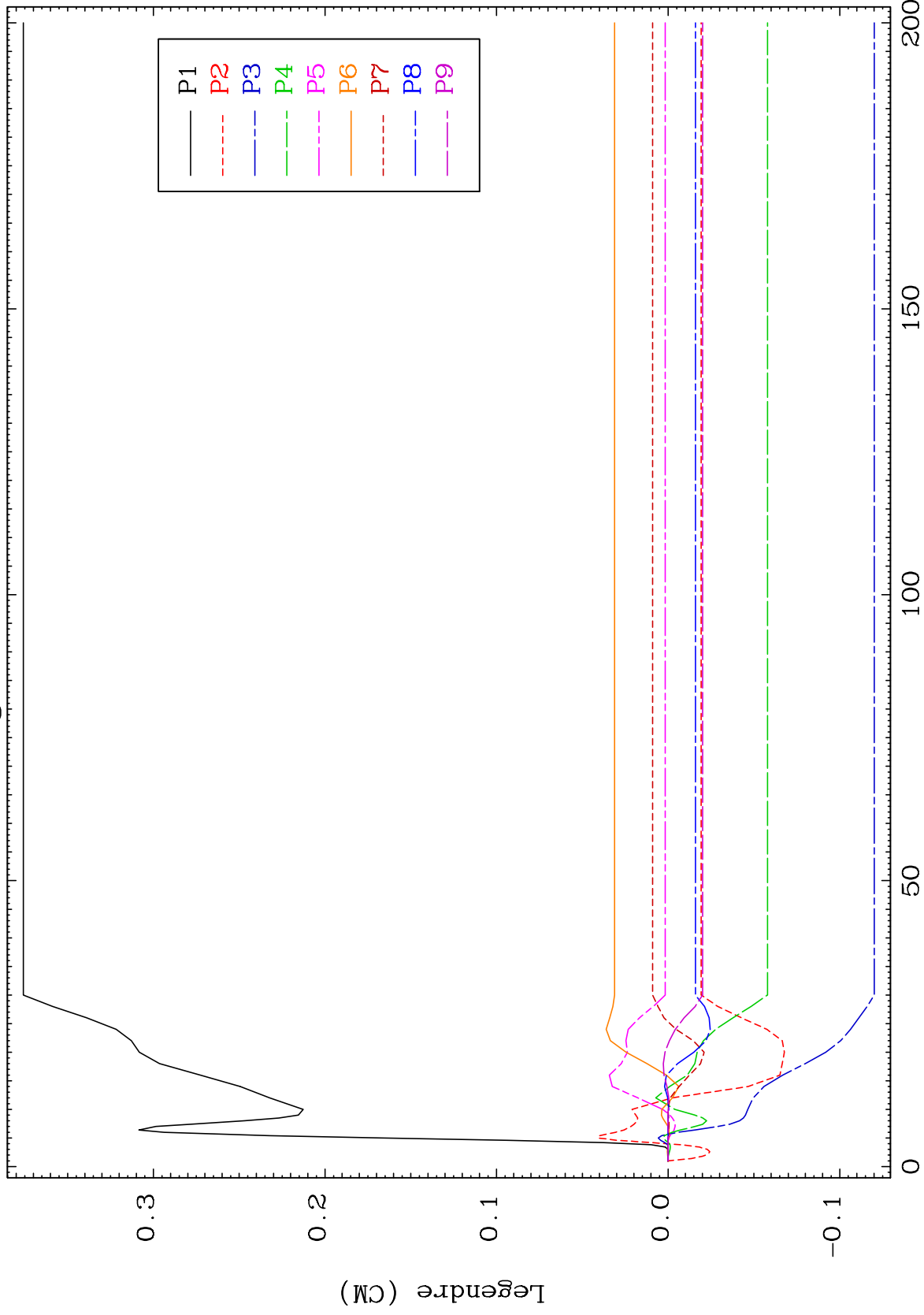




MAT 8713

MT= 69 (n,n') Level
Legendre Coefficients

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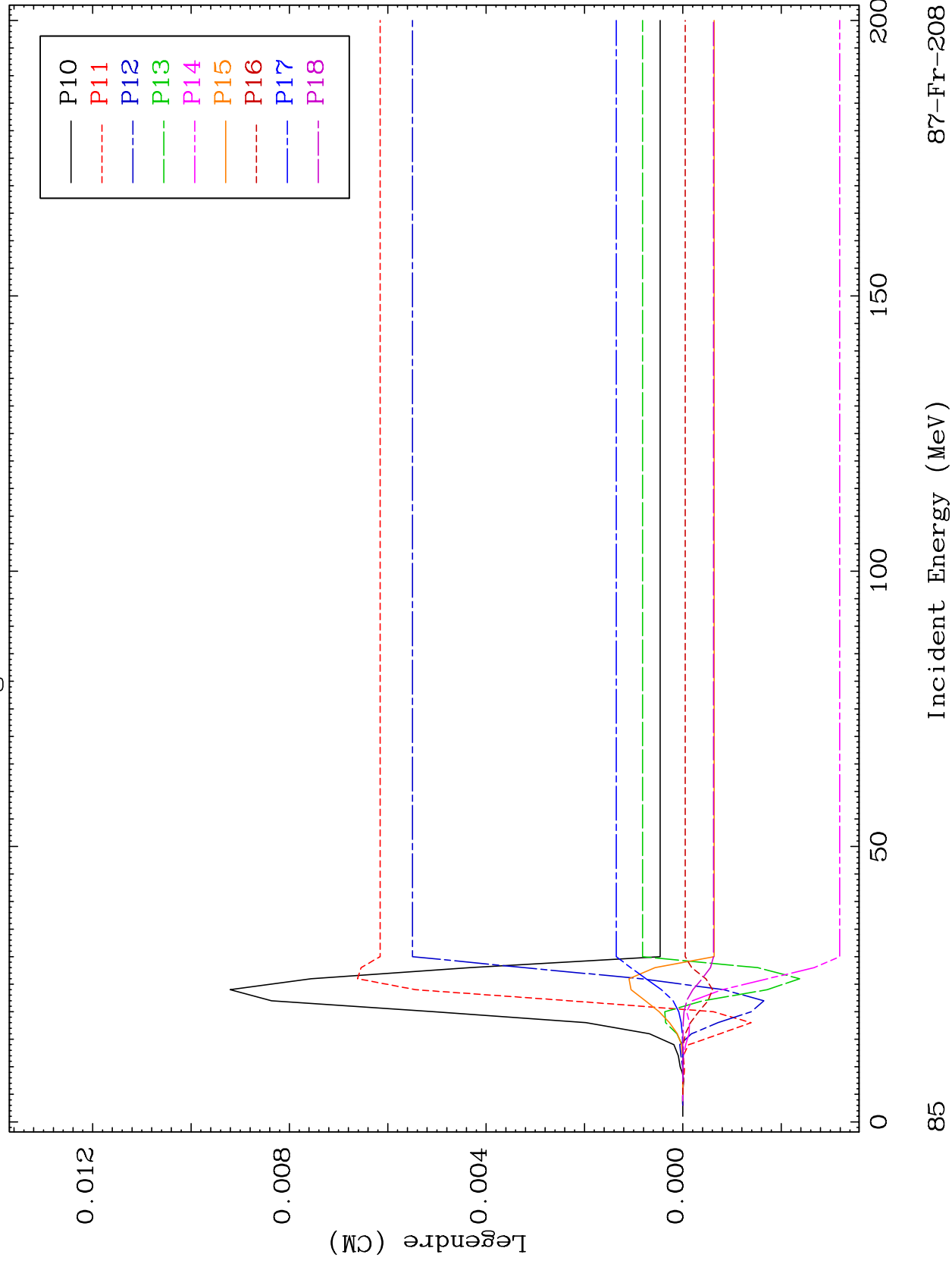
Incident Energy (MeV)

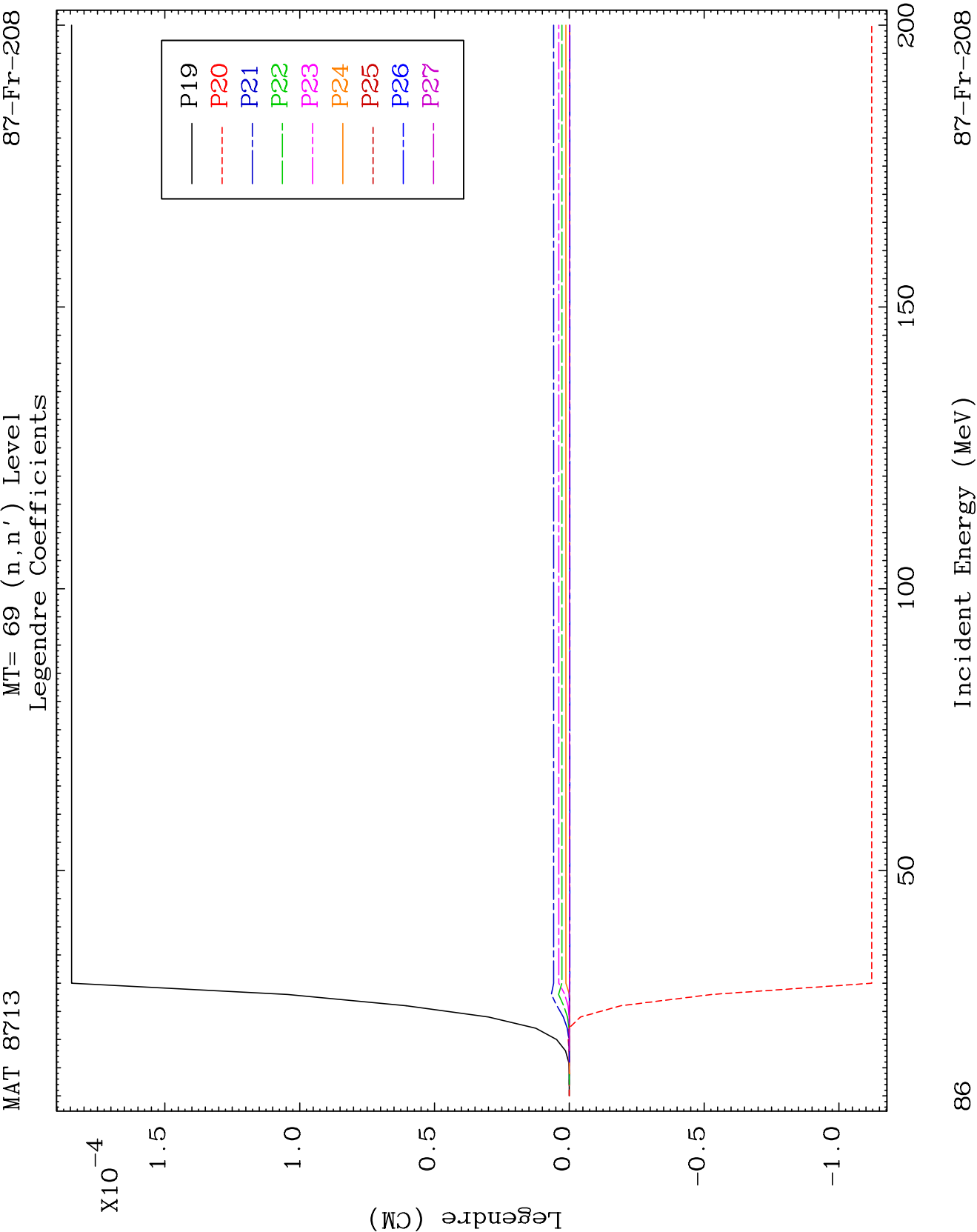
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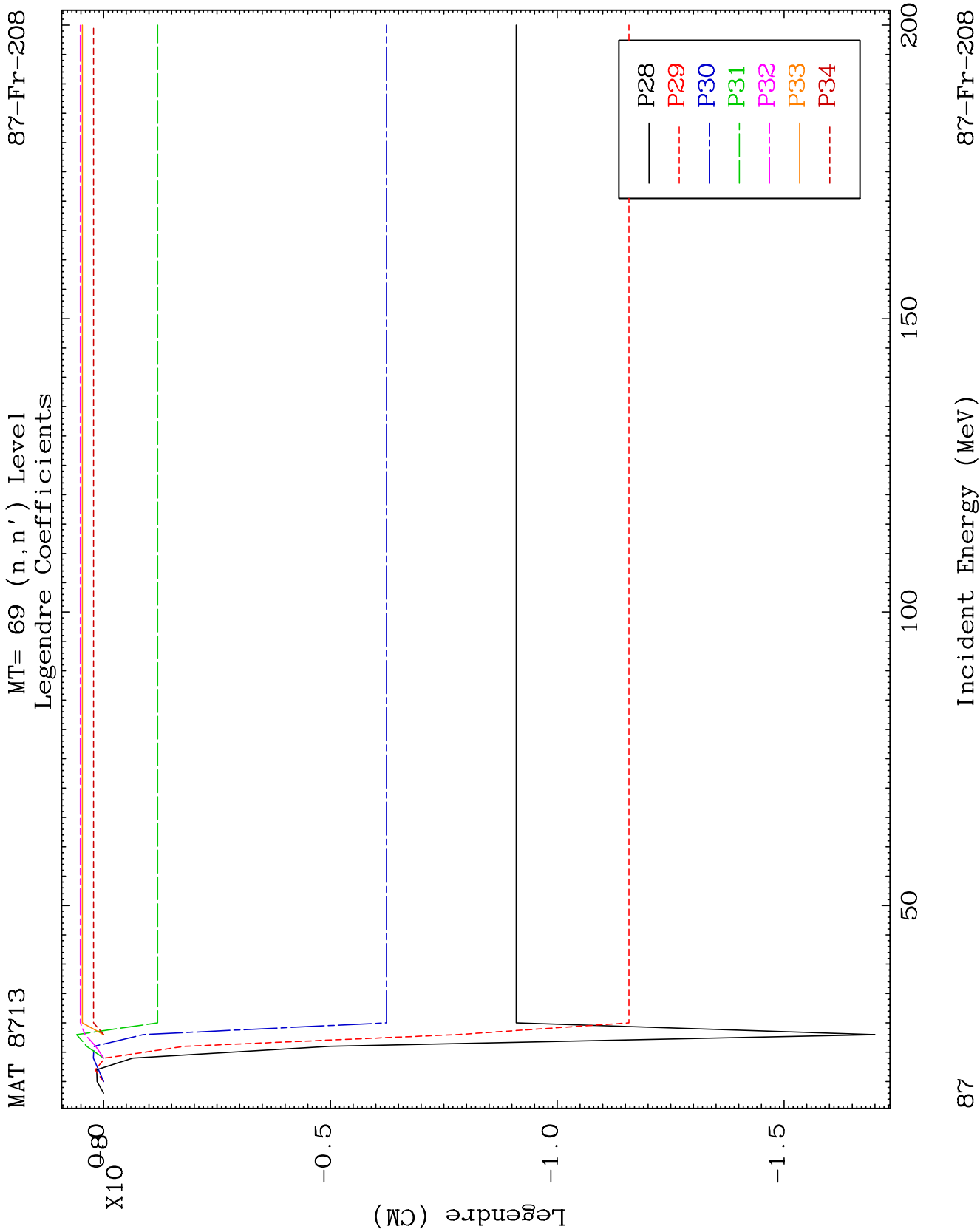
MAT 8713

MT= 69 (n, n') Level
Legendre Coefficients

87-Fr-208



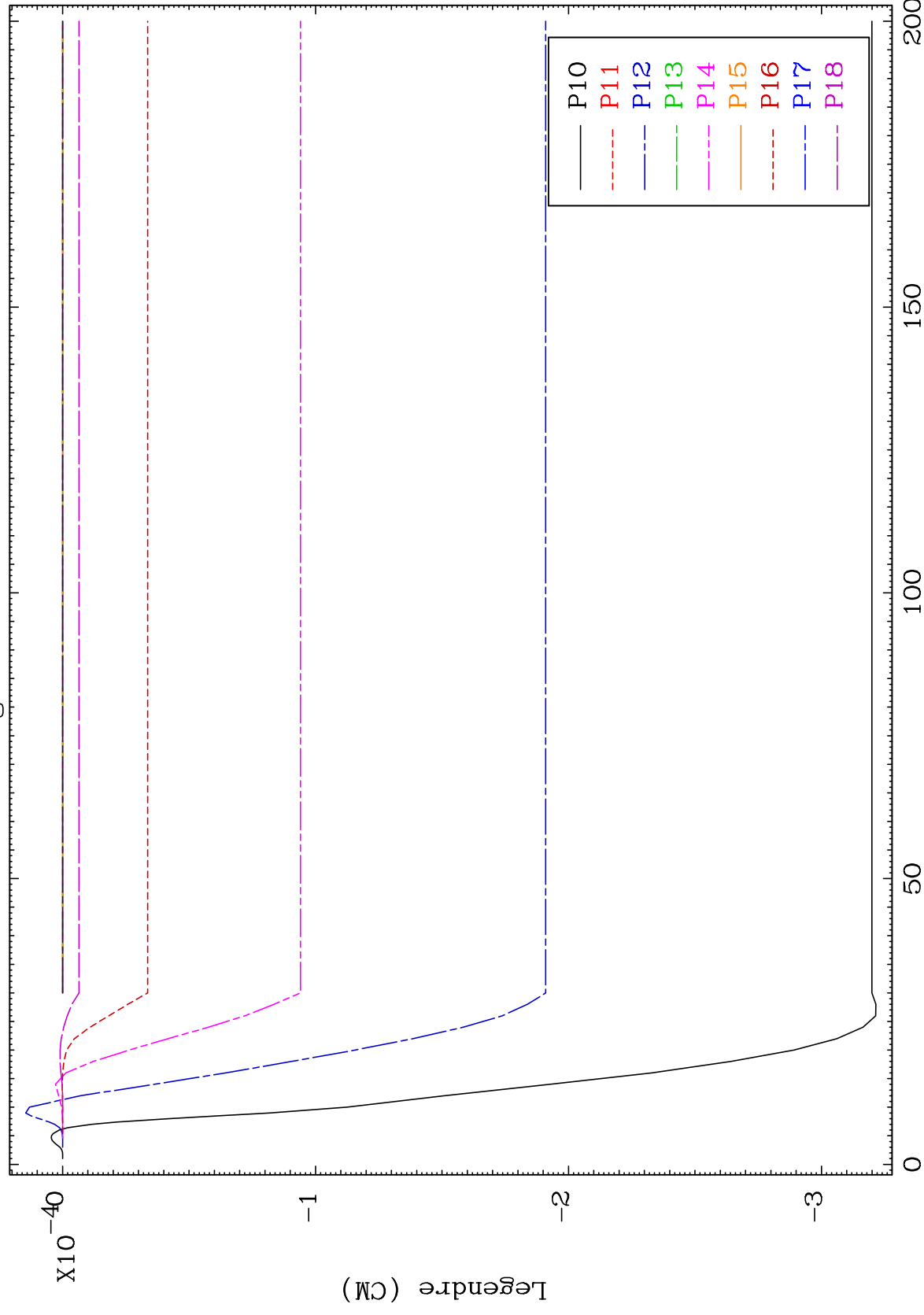




MAT 8713

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MT= 70 (n, n') Level
Legendre Coefficients



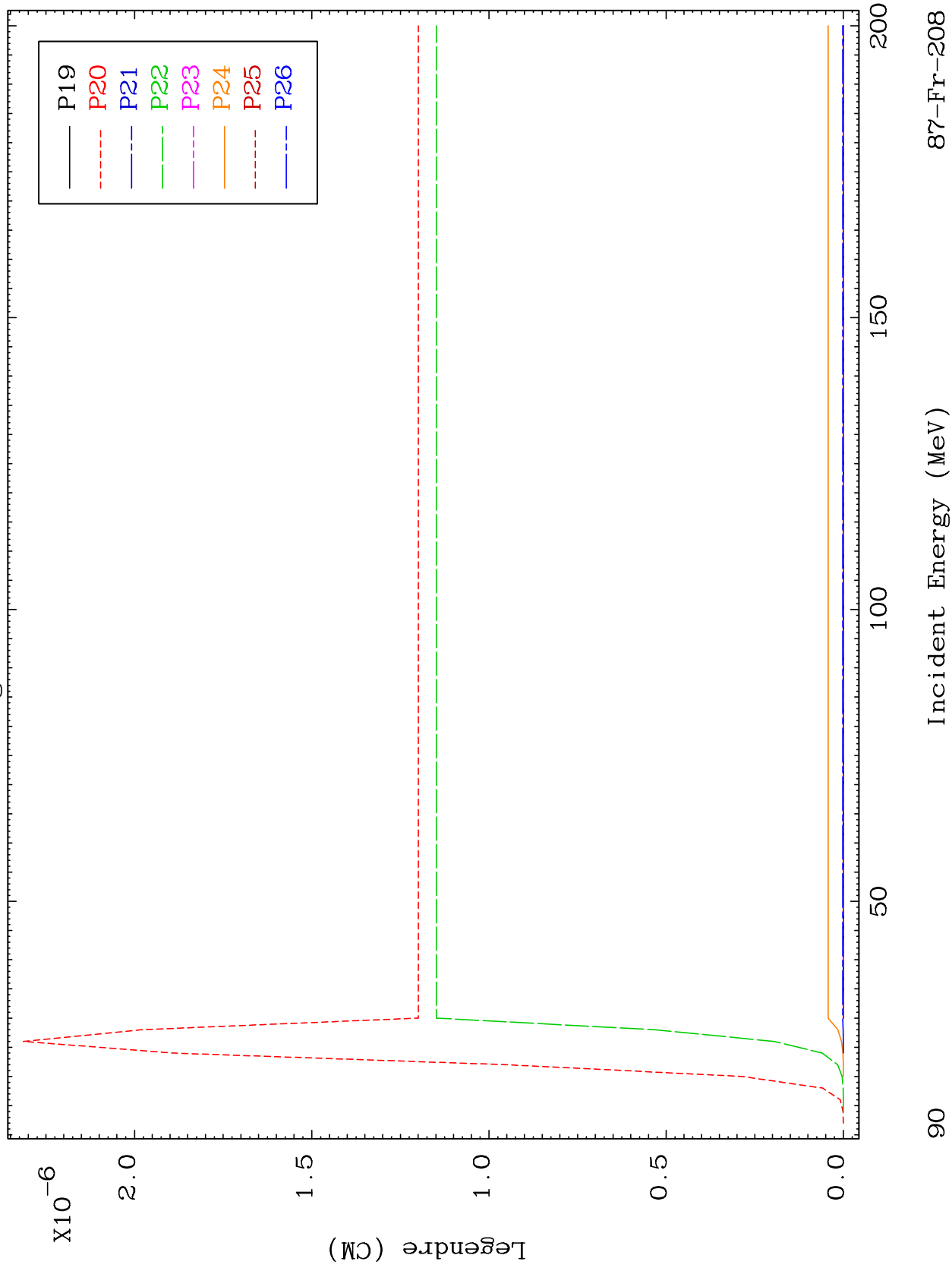
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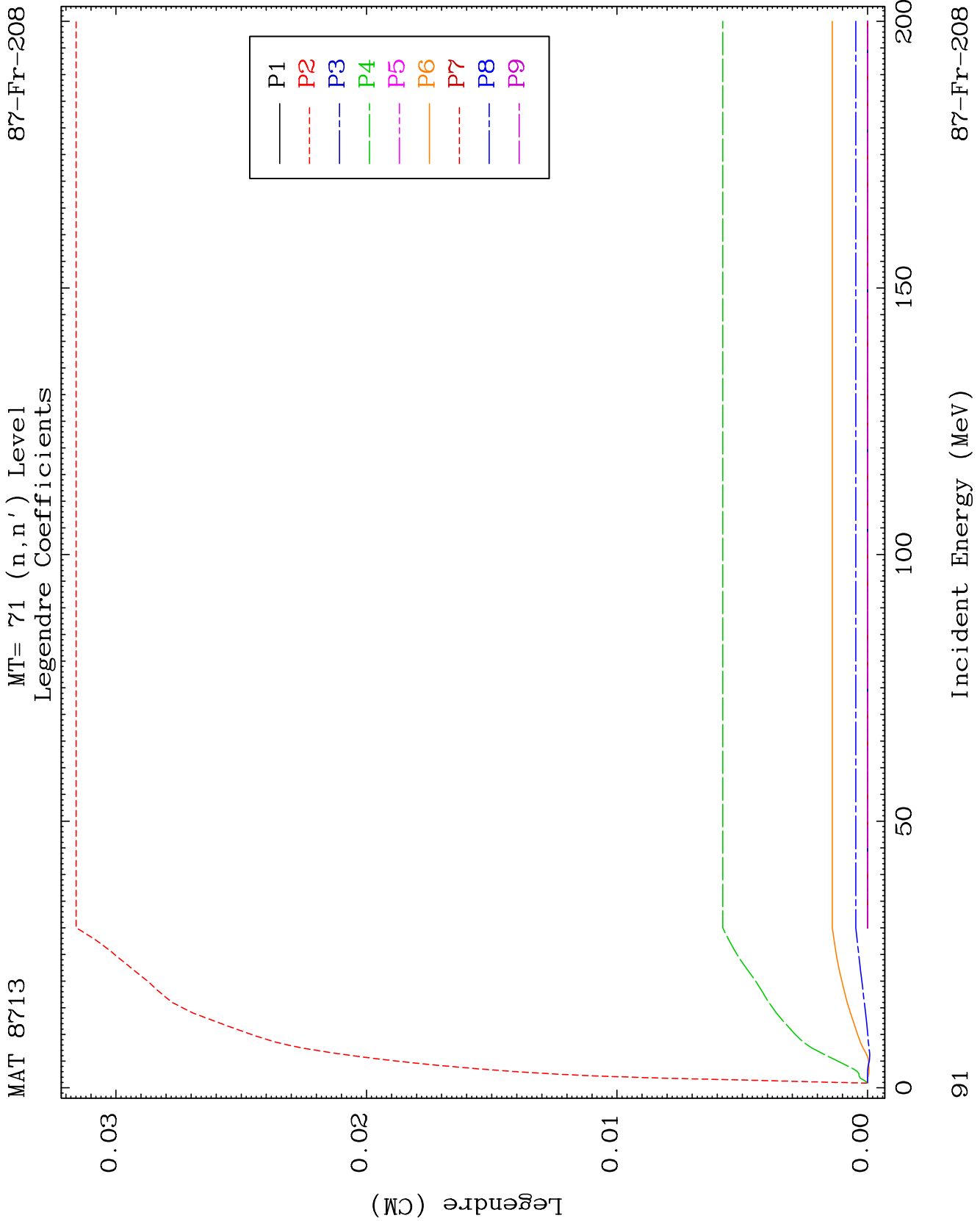
Incident Energy (MeV)

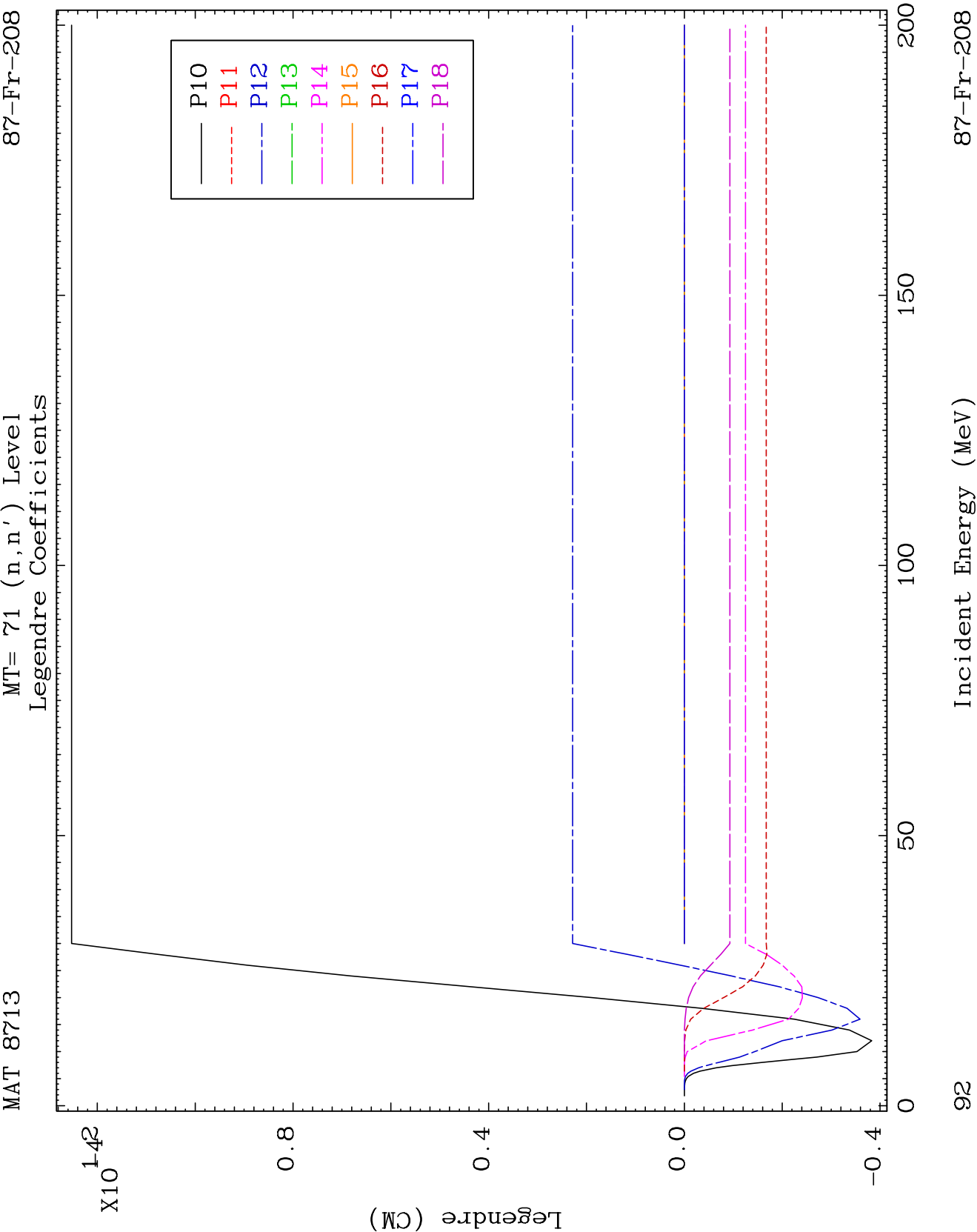
MAT 8713

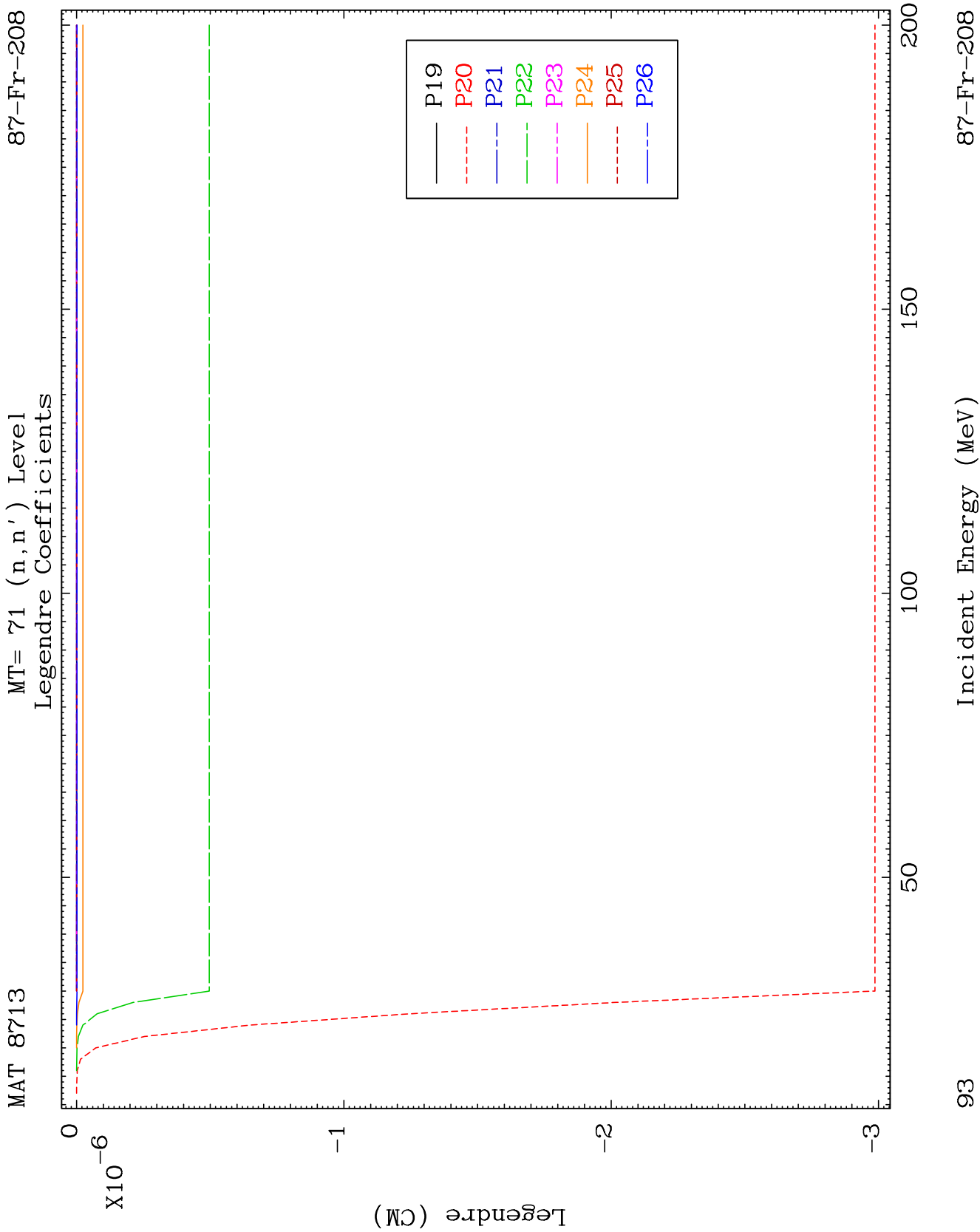
MT= 70 (n, n') Level
Legendre Coefficients

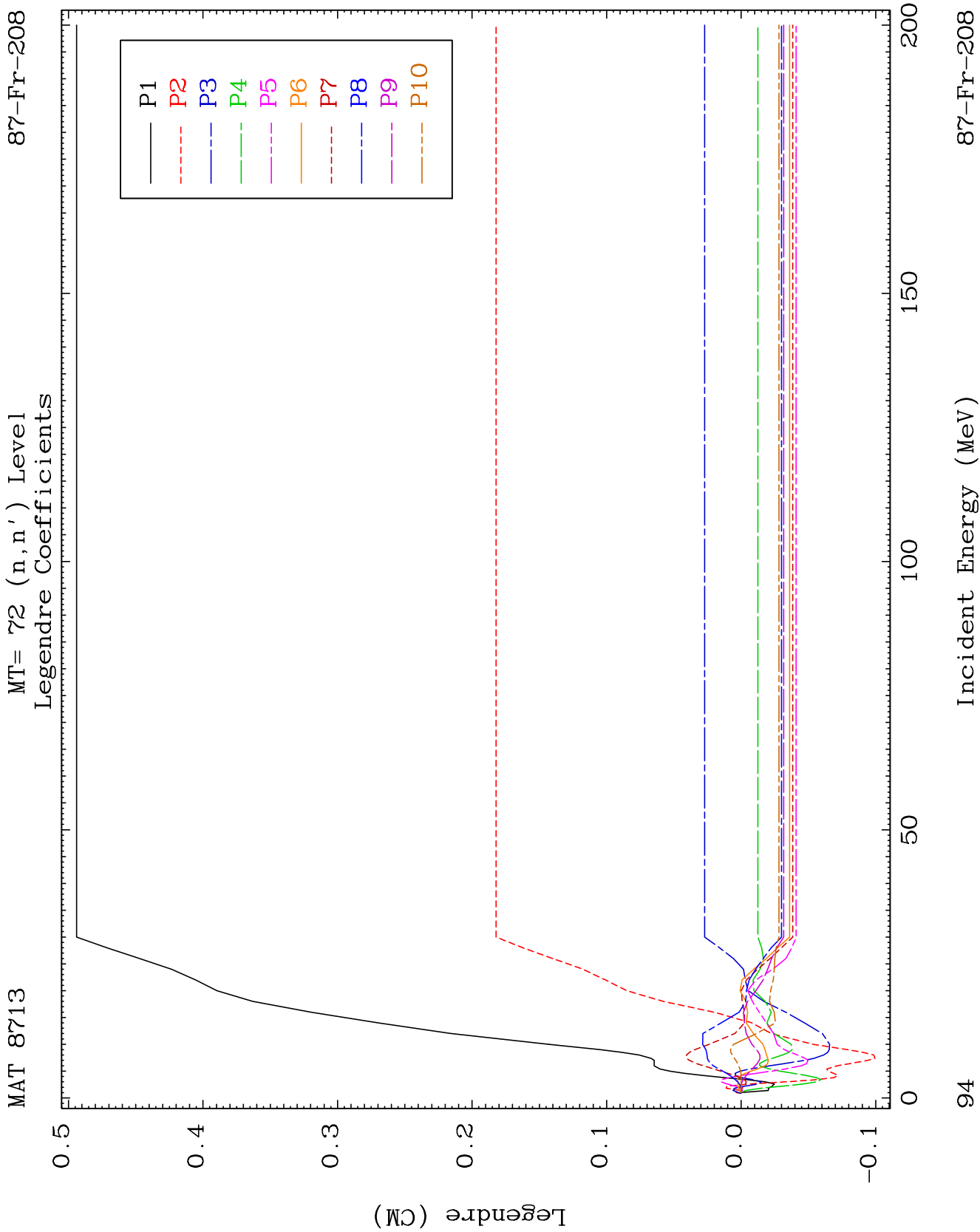
87-Fr-208

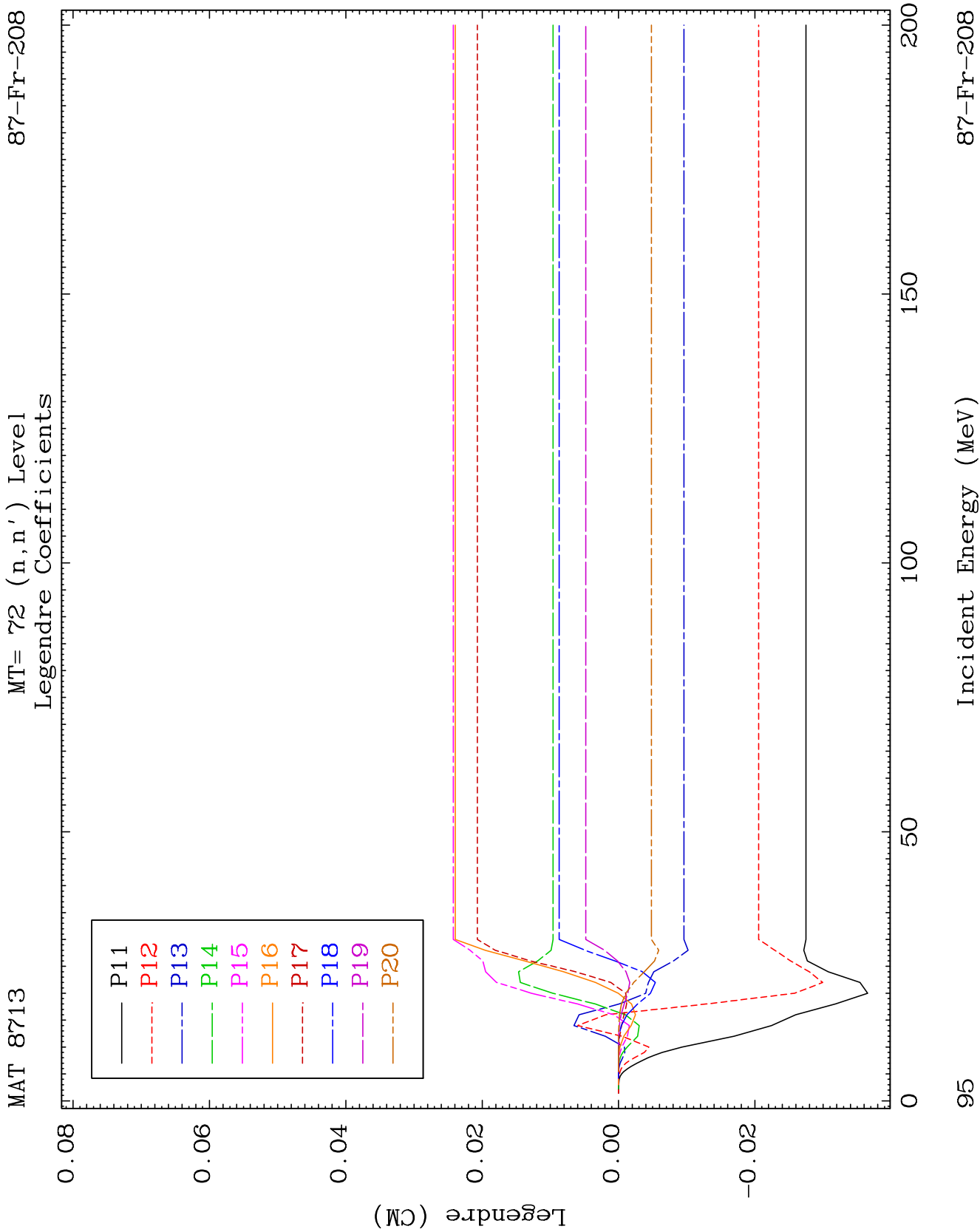








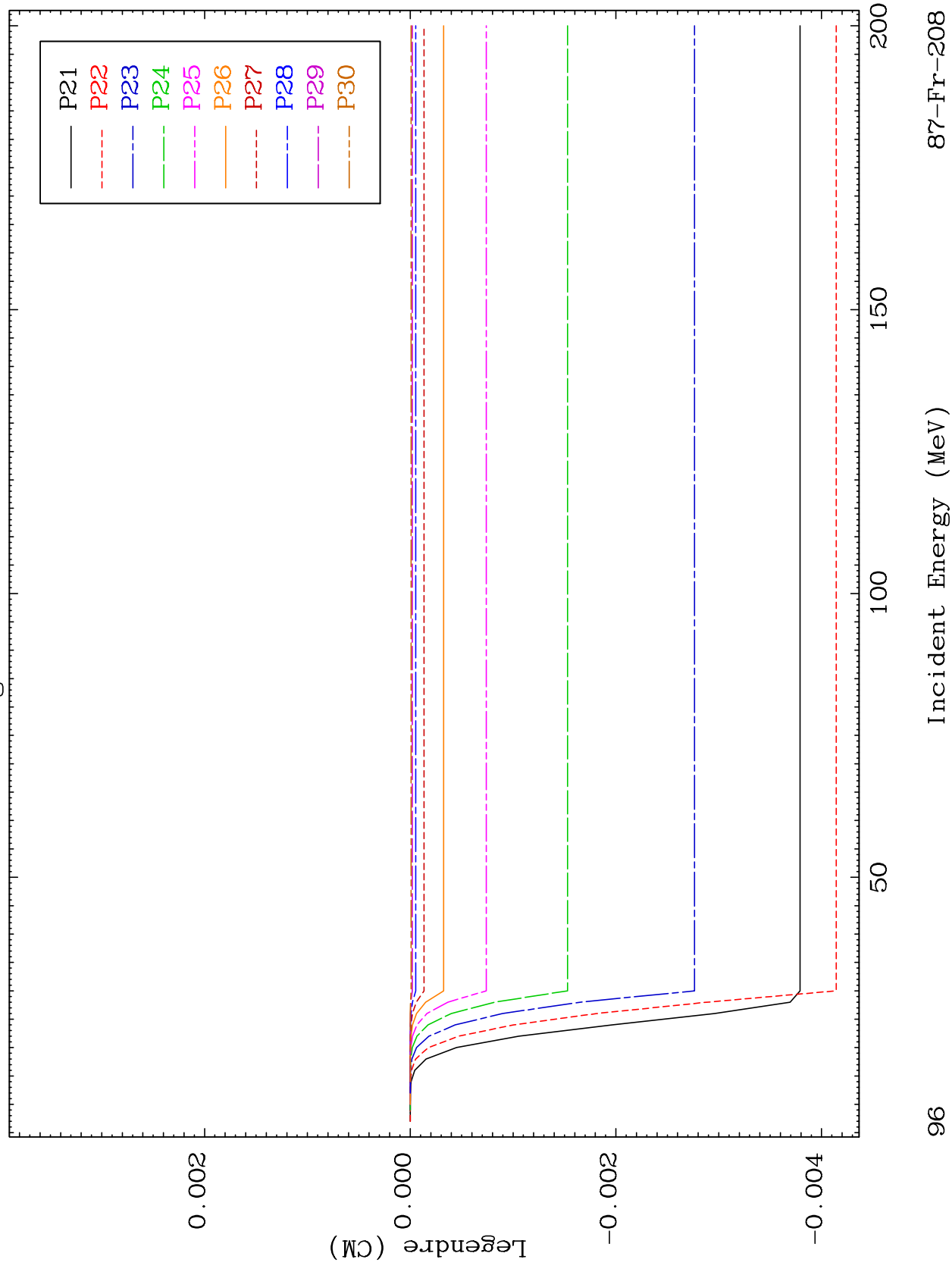


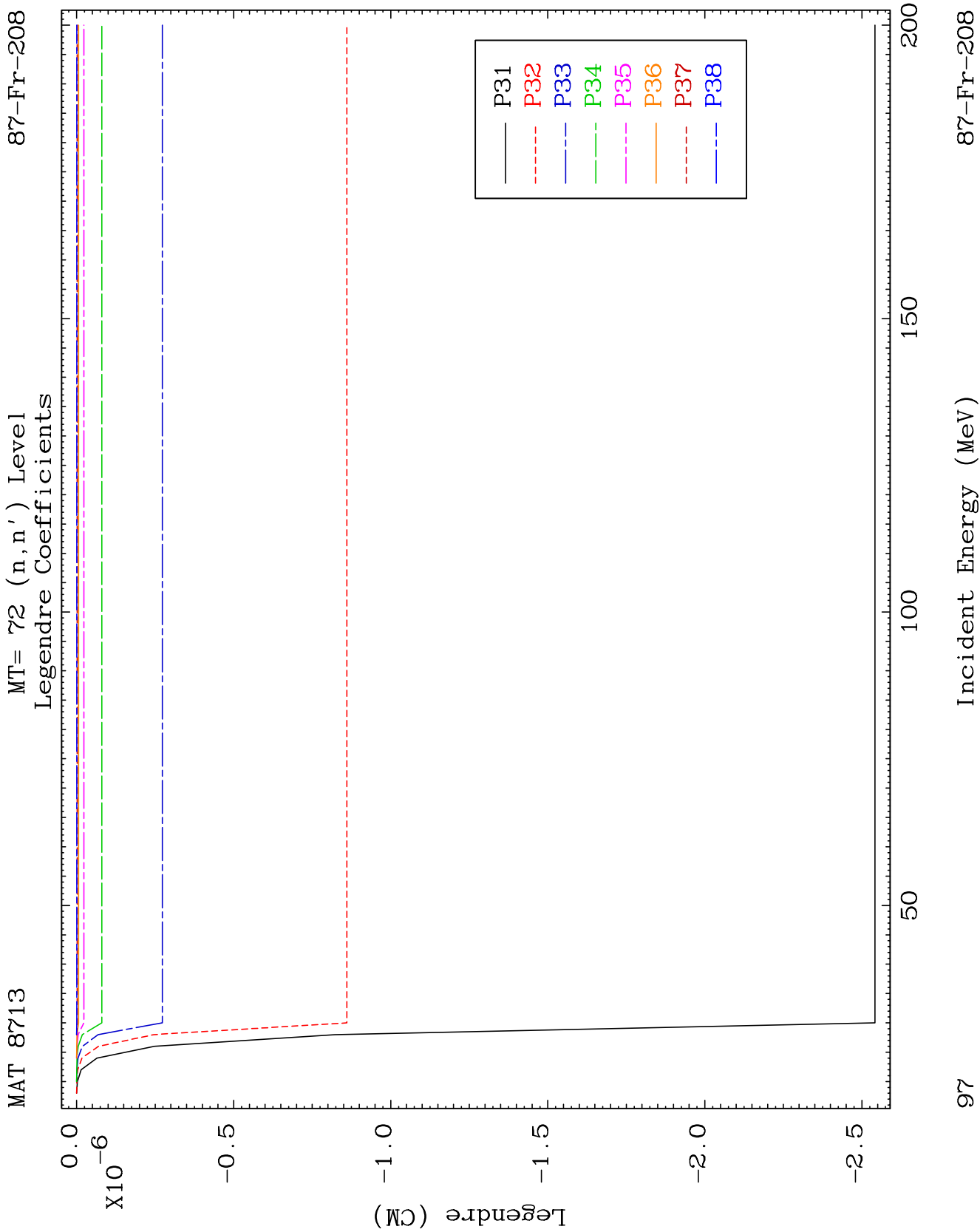


MAT 8713

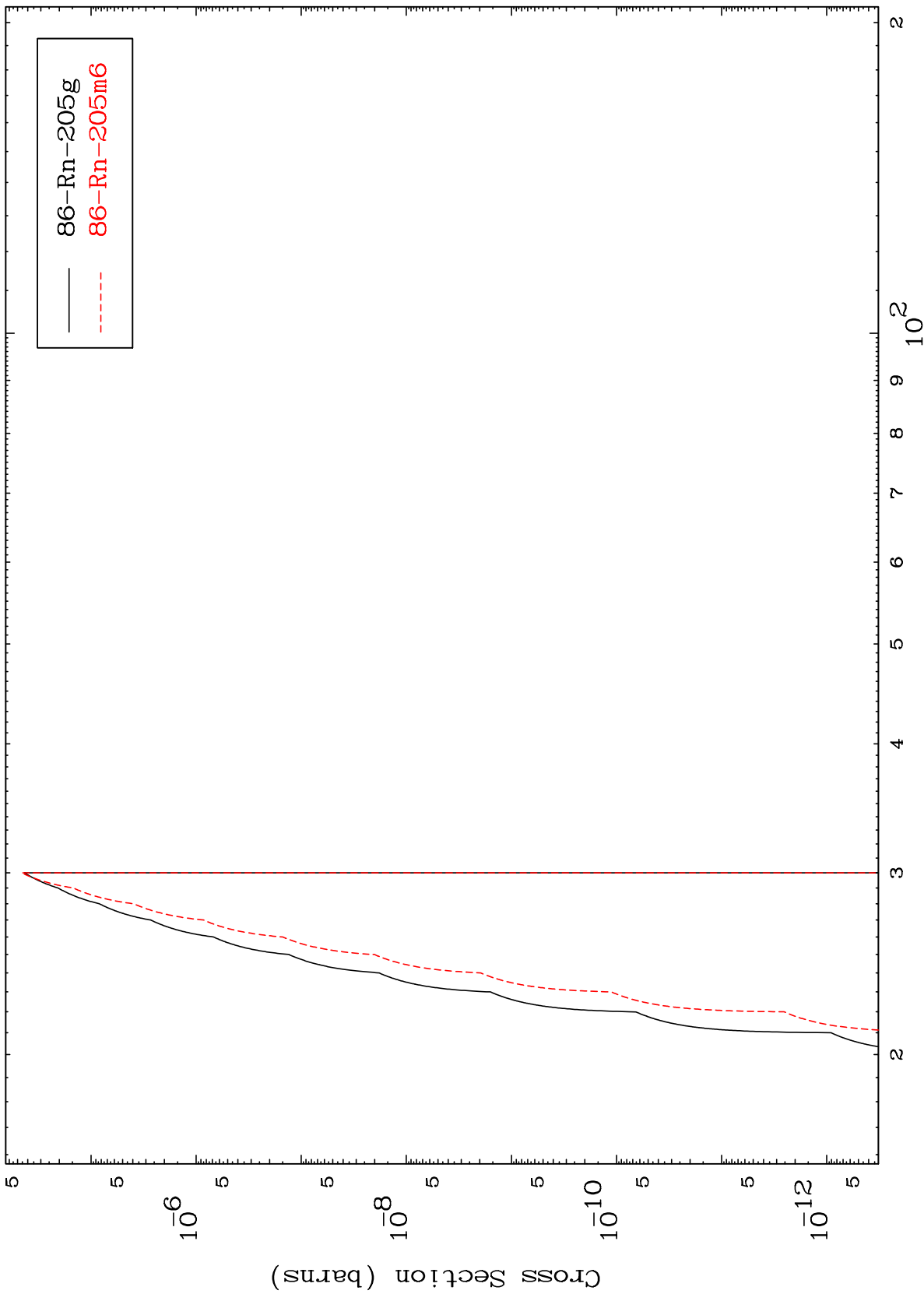
MT= 72 (n, n') Level
Legendre Coefficients

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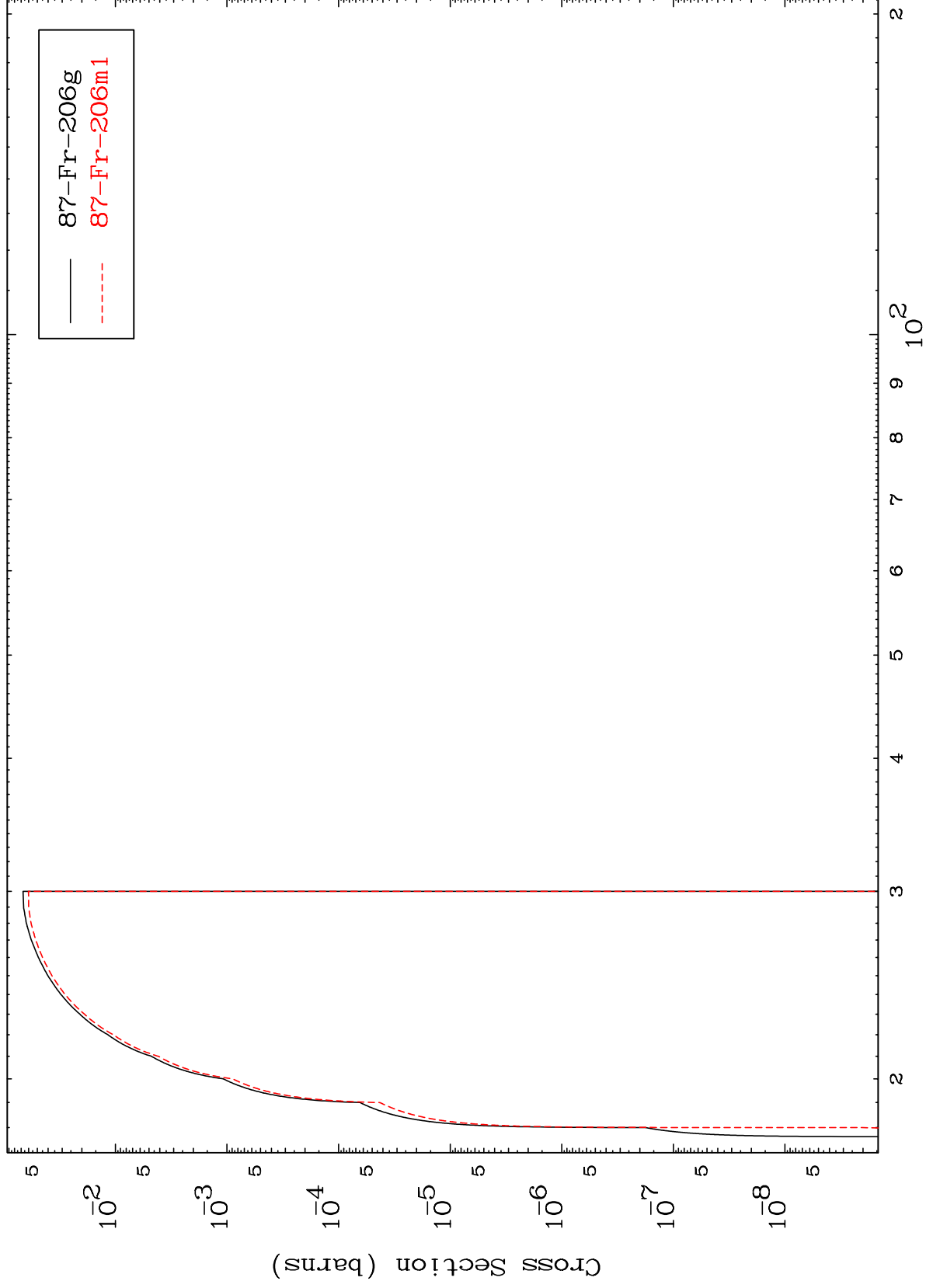
Radionuclide Production Cross Section



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(n,3n)
Radionuclide Production Cross Section



99

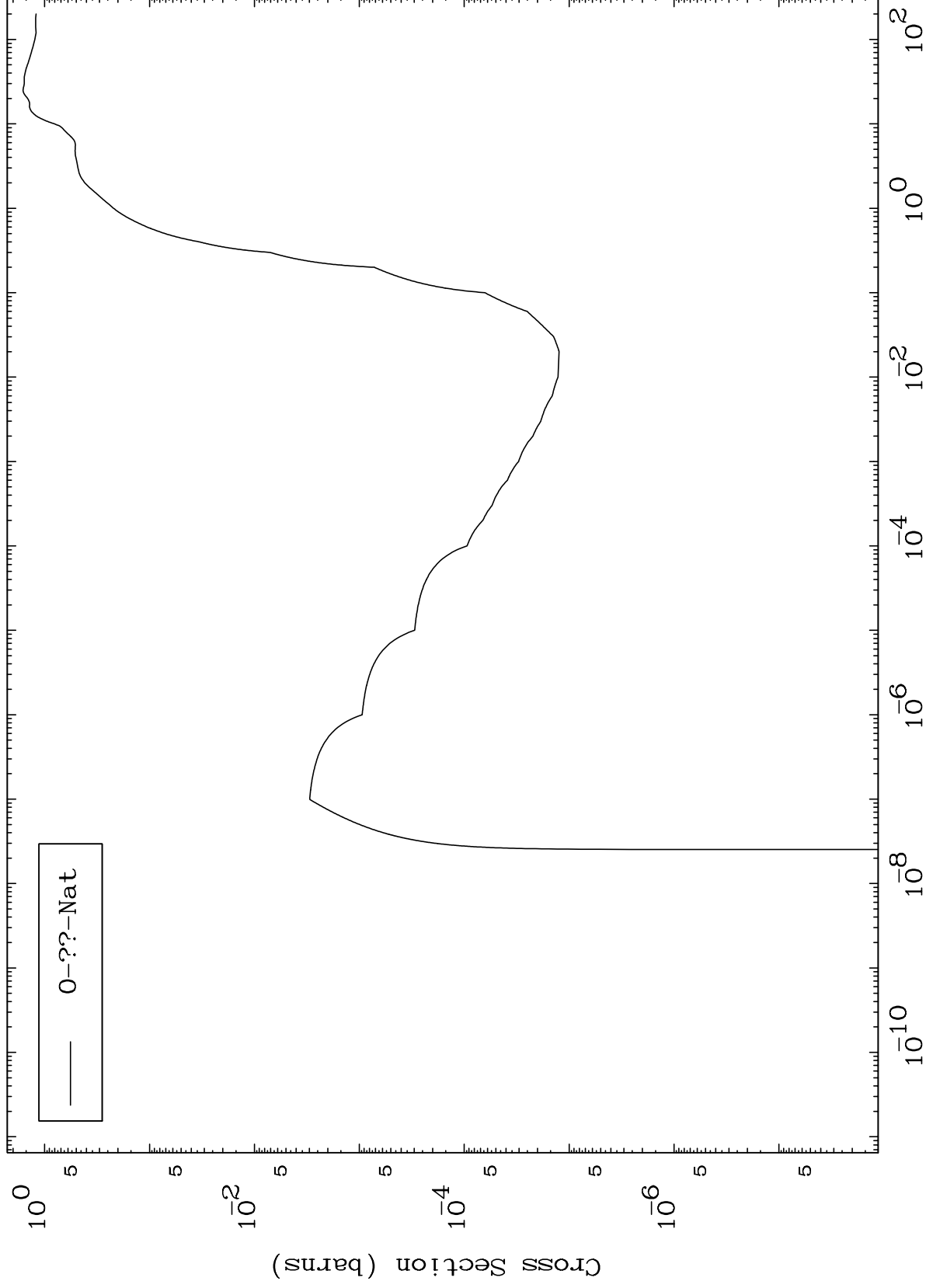
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Fission

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Radionuclide Production Cross Section



100

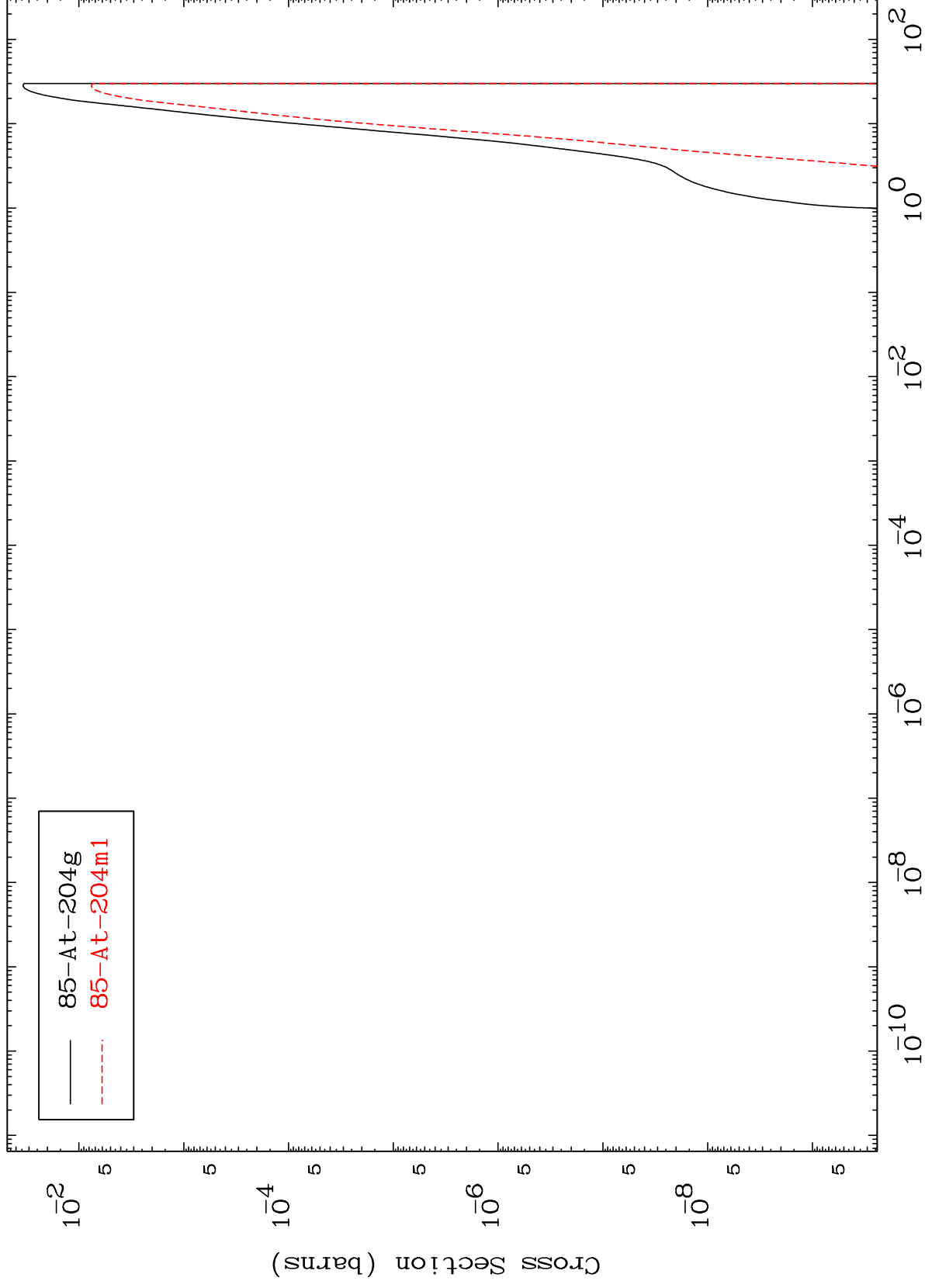
Incident Energy (MeV)

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87-Fr-208

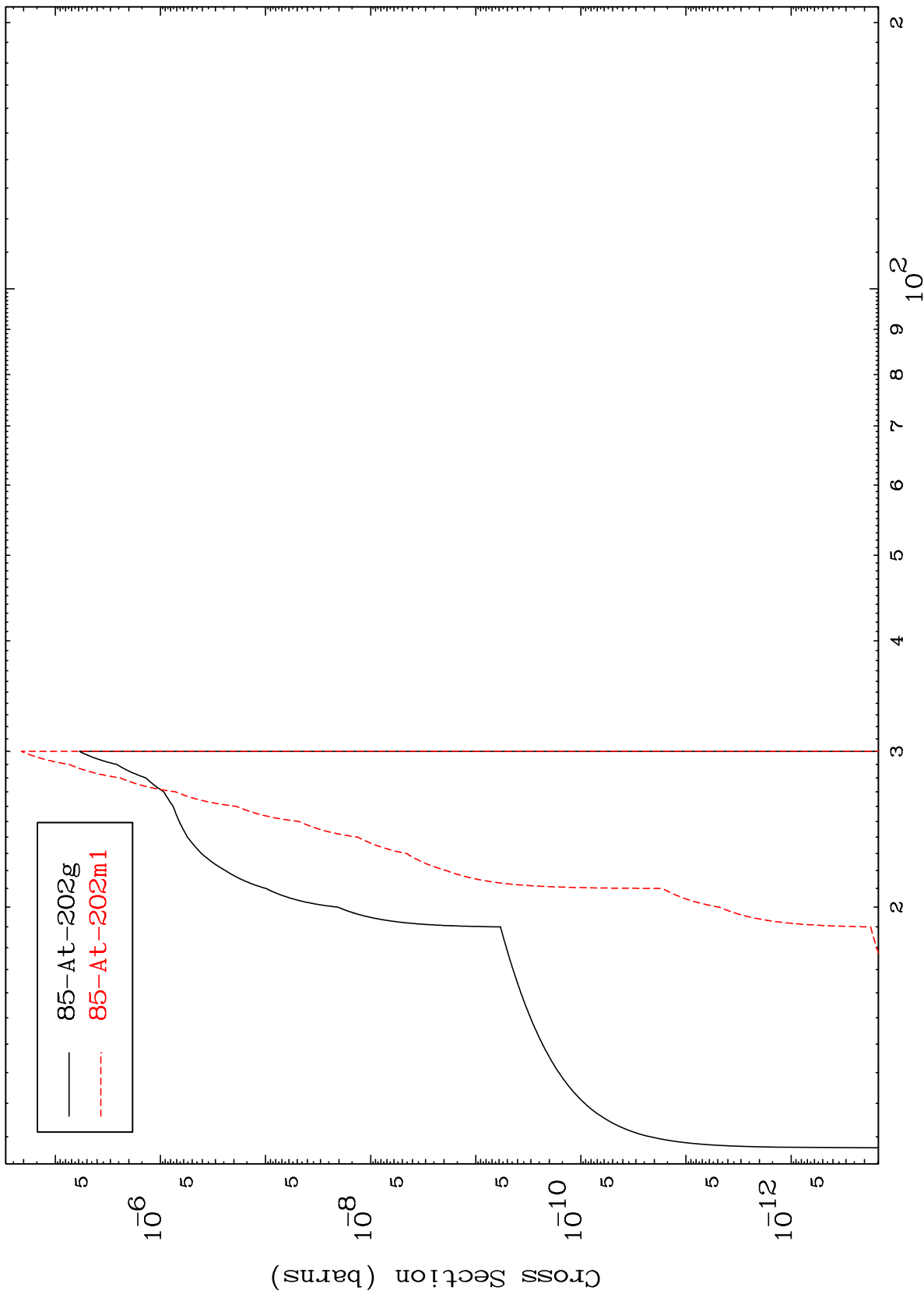
$(n,n') \alpha$
Radionuclide Production Cross Section



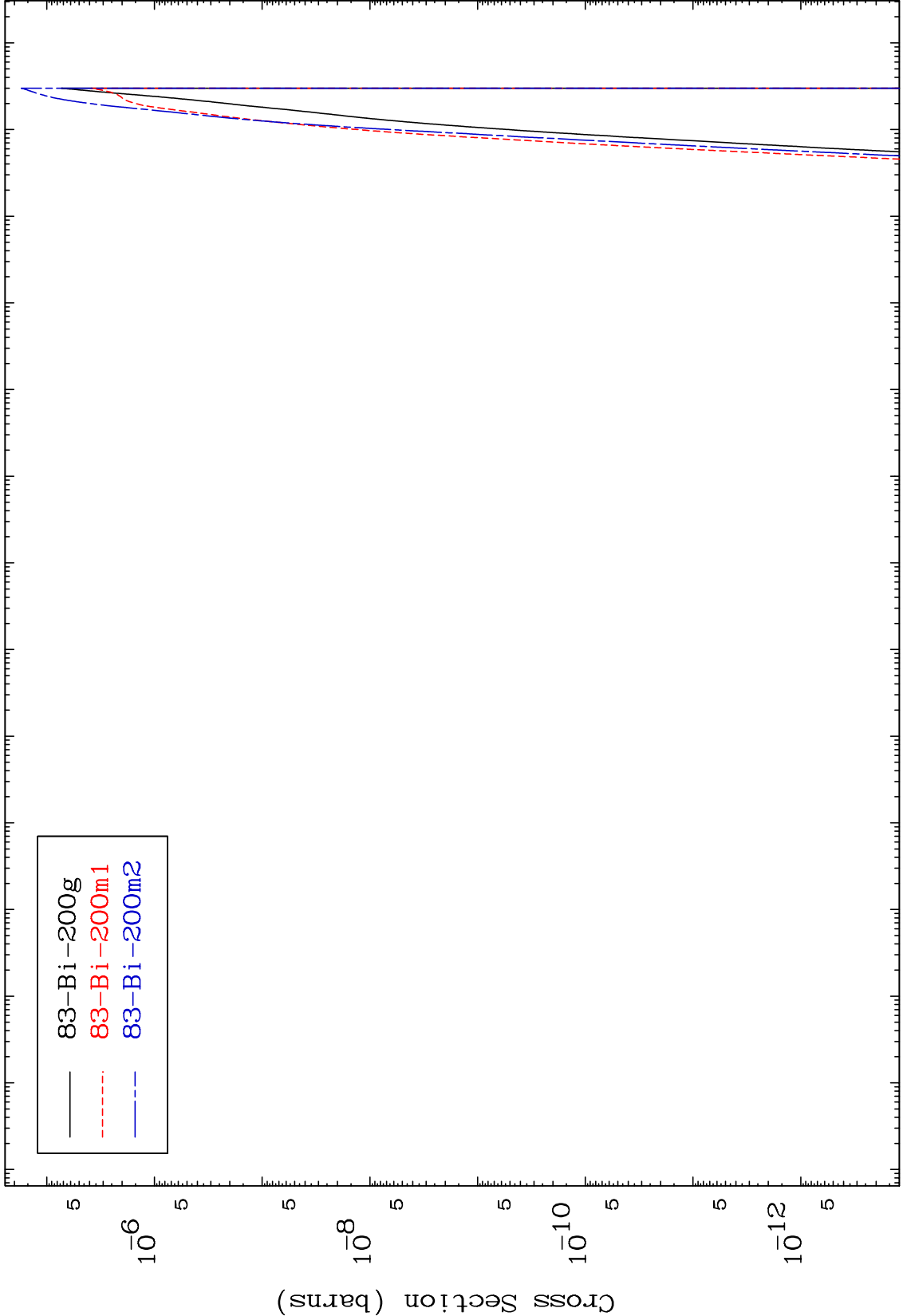
101

87-Fr-208

Radionuclide Production Cross Section



Radionuclide Production Cross Section

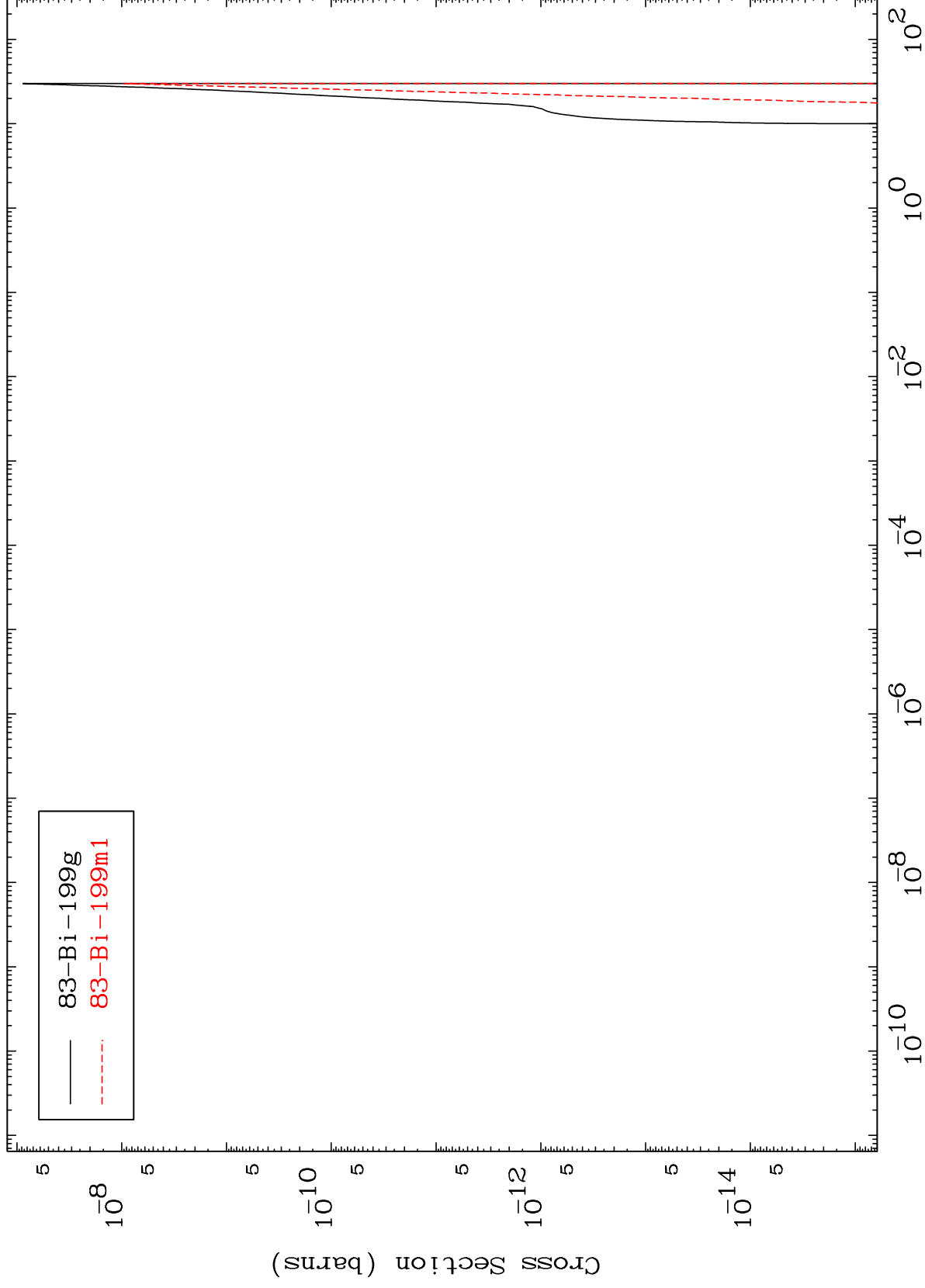


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(n,2n) 2 α

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Radionuclide Production Cross Section



104

Incident Energy (MeV)

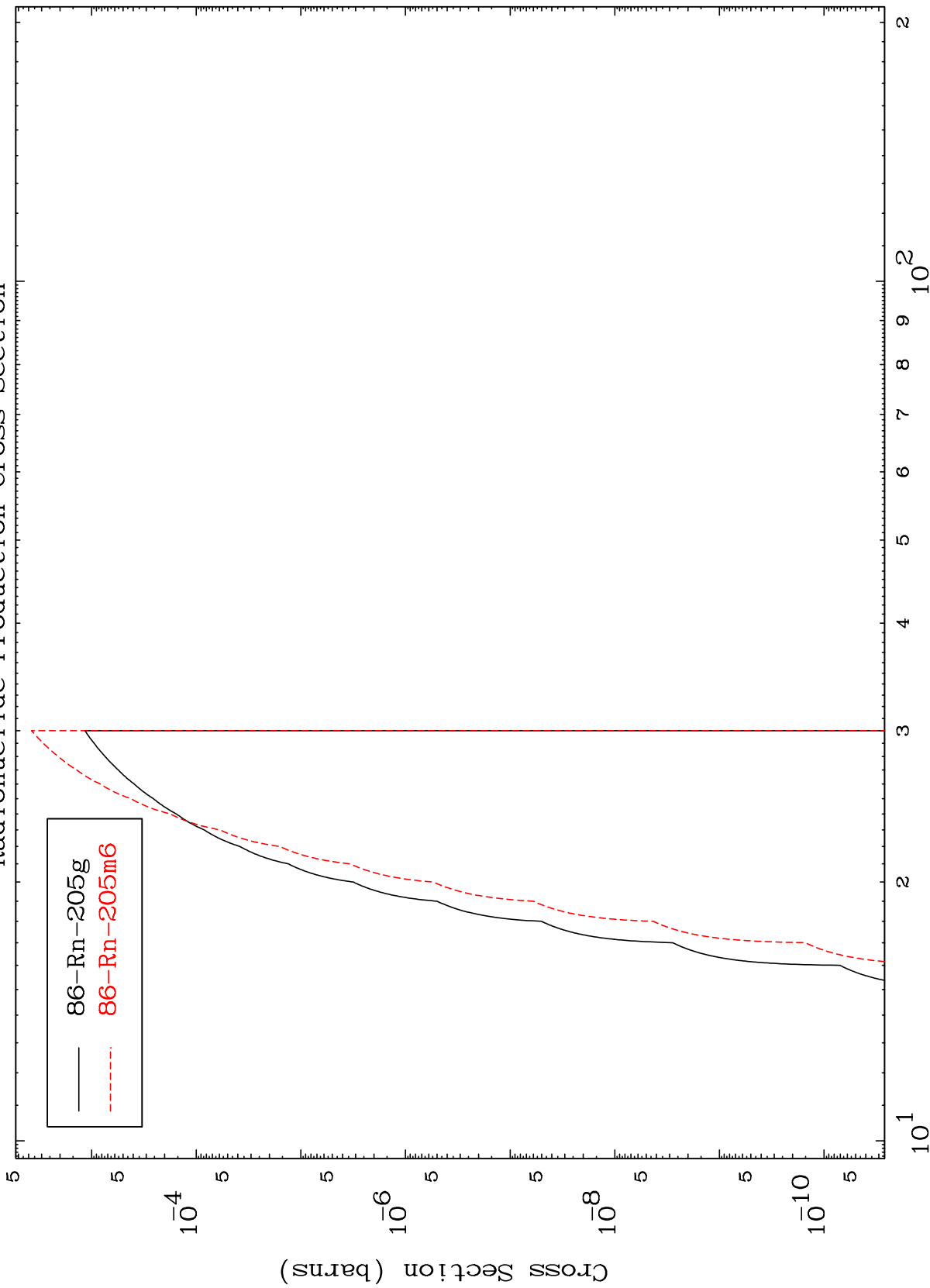
87-Fr-208

MAT 8713

87-Fr-208

(n,n') t

Radionuclide Production Cross Section



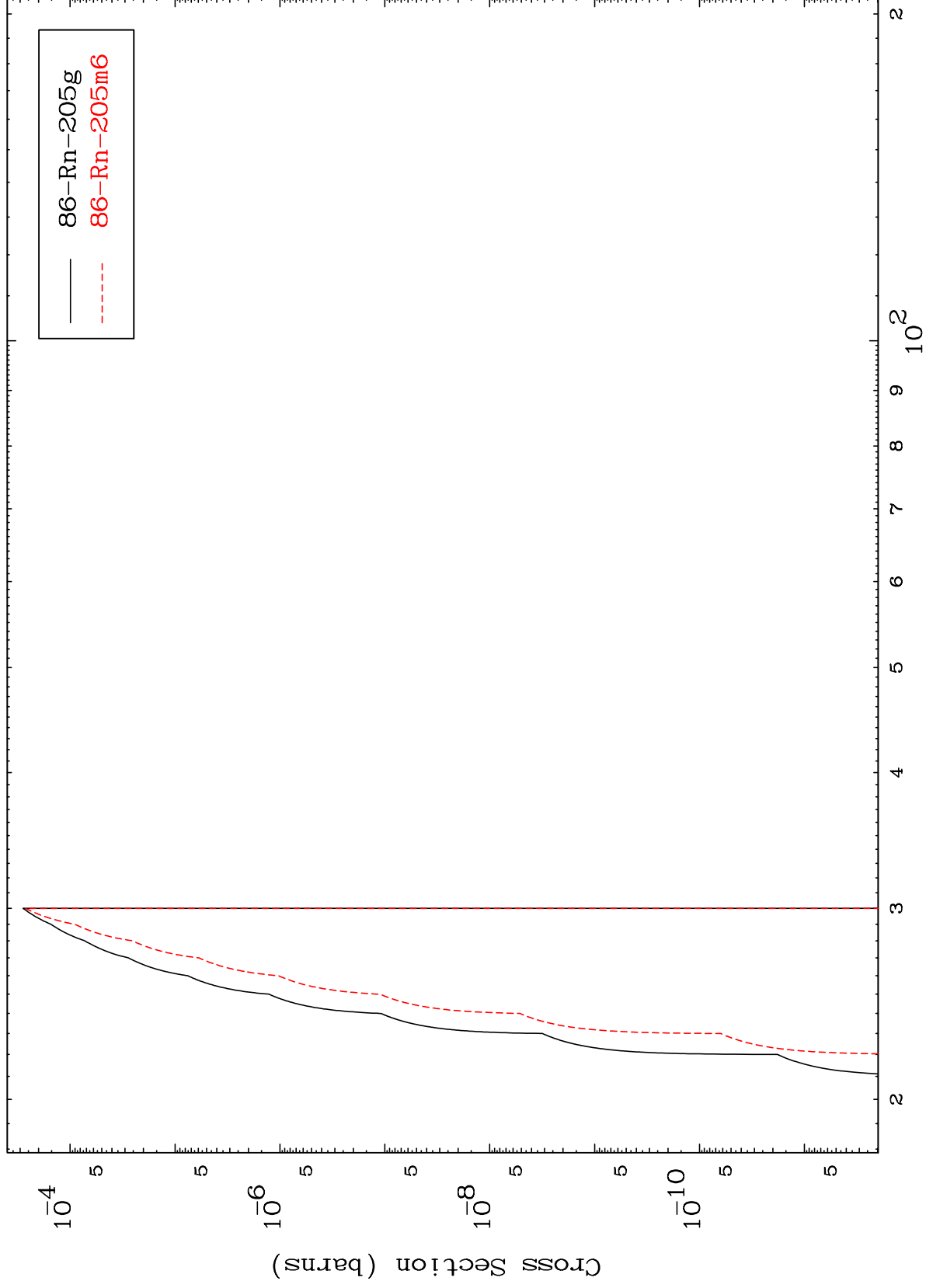
Incident Energy (MeV)

87-Fr-208

MAT 8713

87-Fr-208

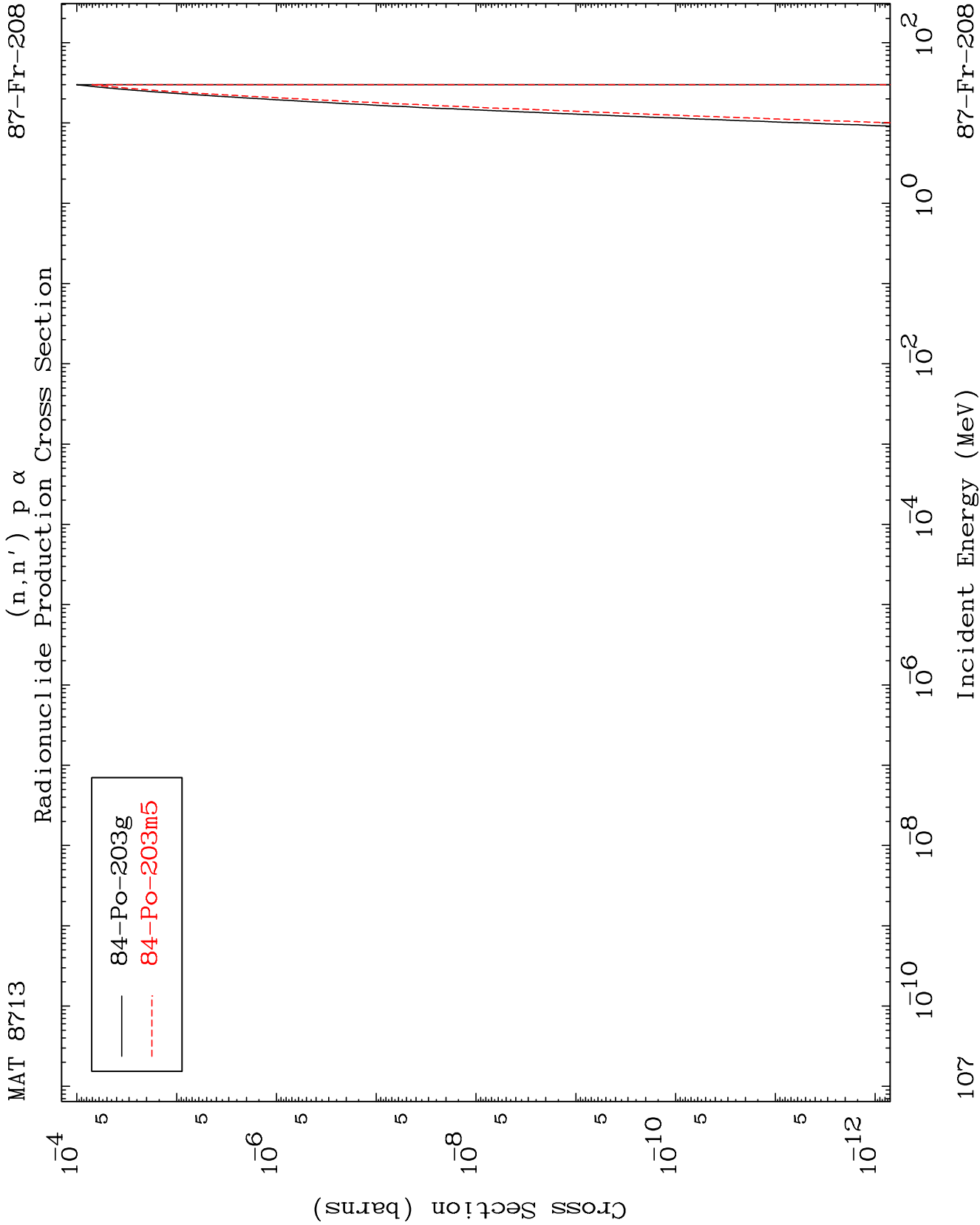
(n,3n) p
Radionuclide Production Cross Section



106

Incident Energy (MeV)

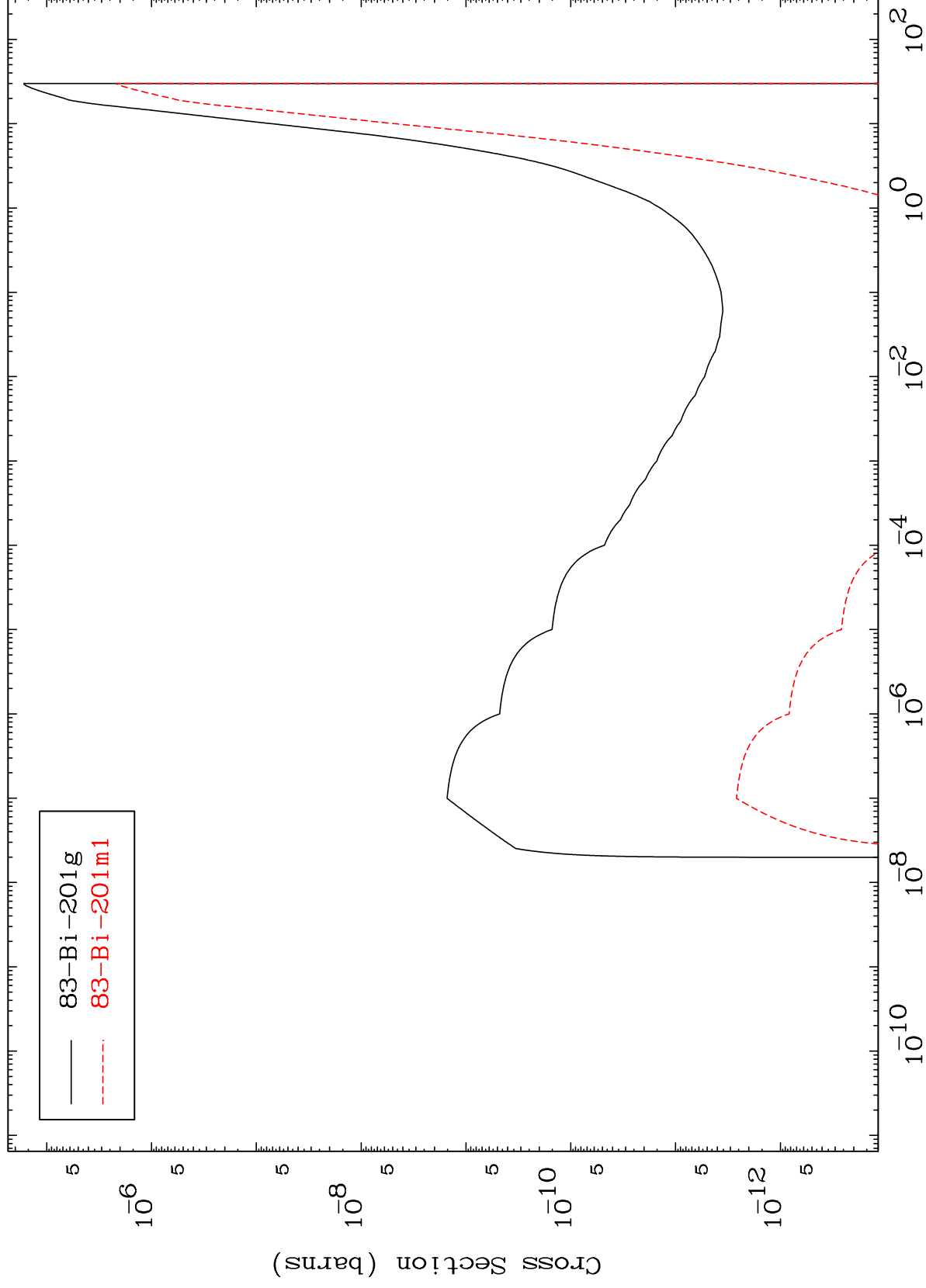
87-Fr-208



MAT 8713

87-Fr-208

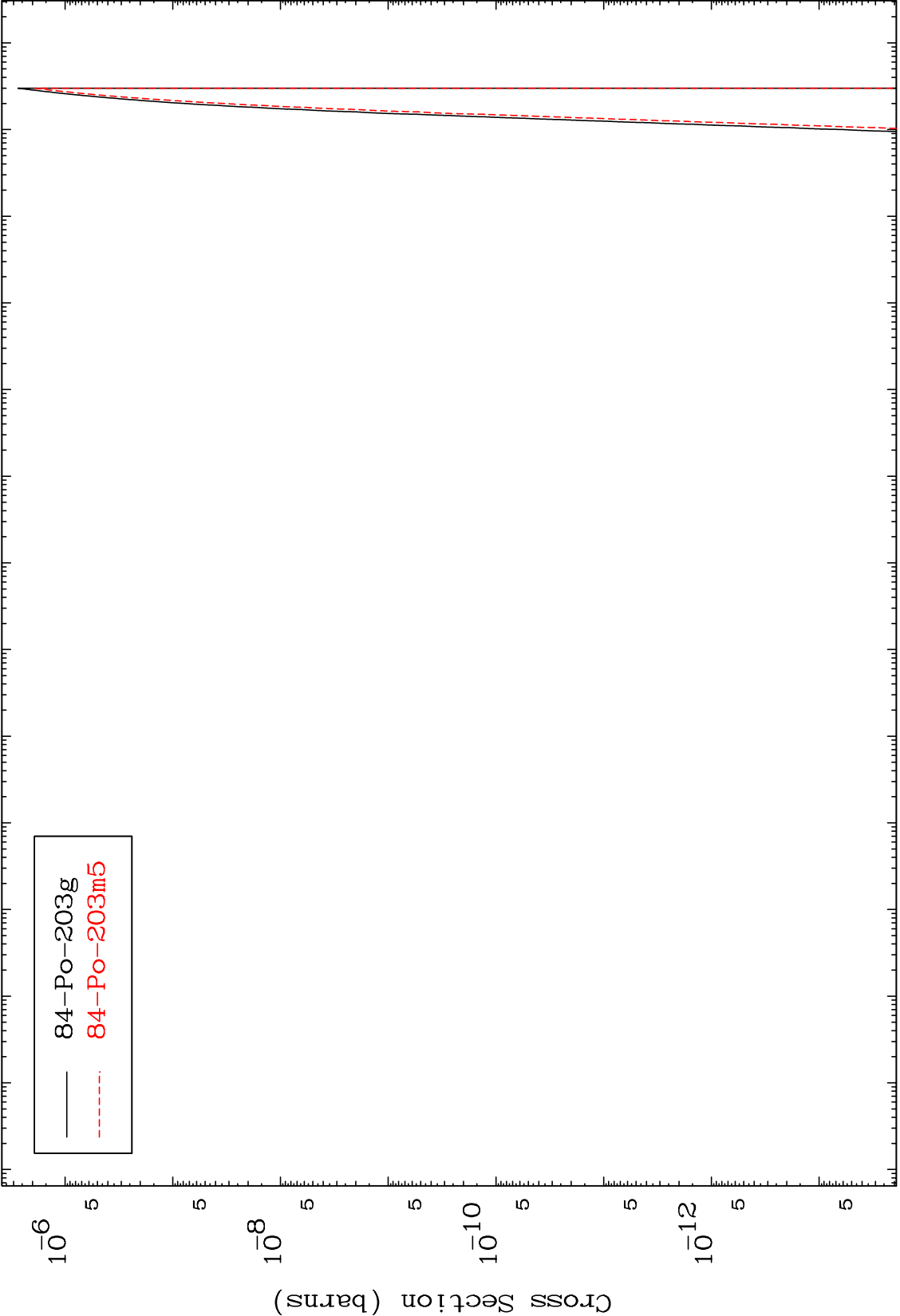
Radionuclide Production Cross Section
(n,2α)



108

87-Fr-208

Radionuclide Production Cross Section



84-Po-203g
84-Po-203m5