

Program EVALPLOT
(Version 2021-1)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail: redcullen1@comcast.net
Web: redcullen1.net/HOMEPAGE.NEW

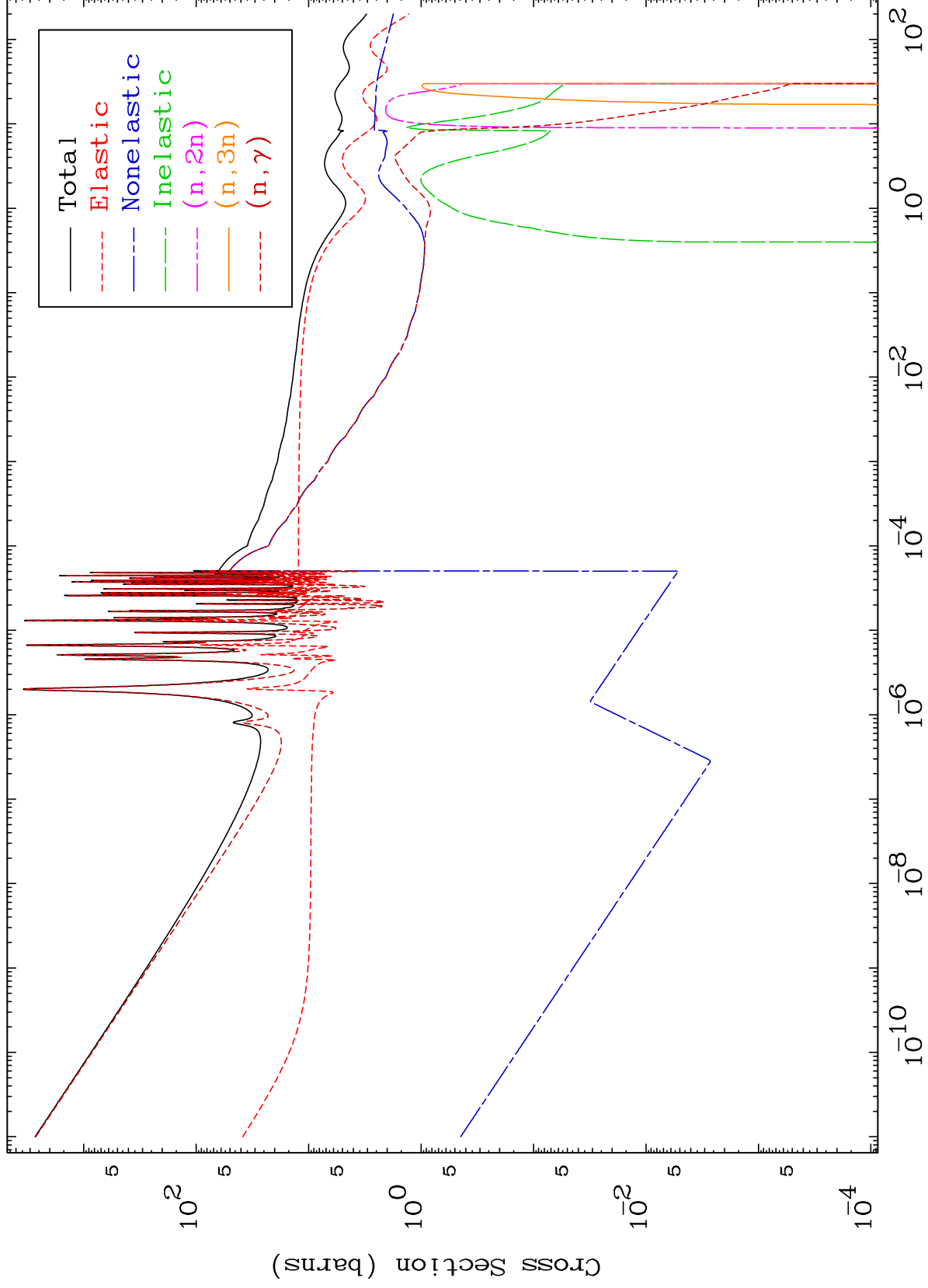
Press Mouse Button to Start

MAT 8722

Neutron Major

87-Fr-211

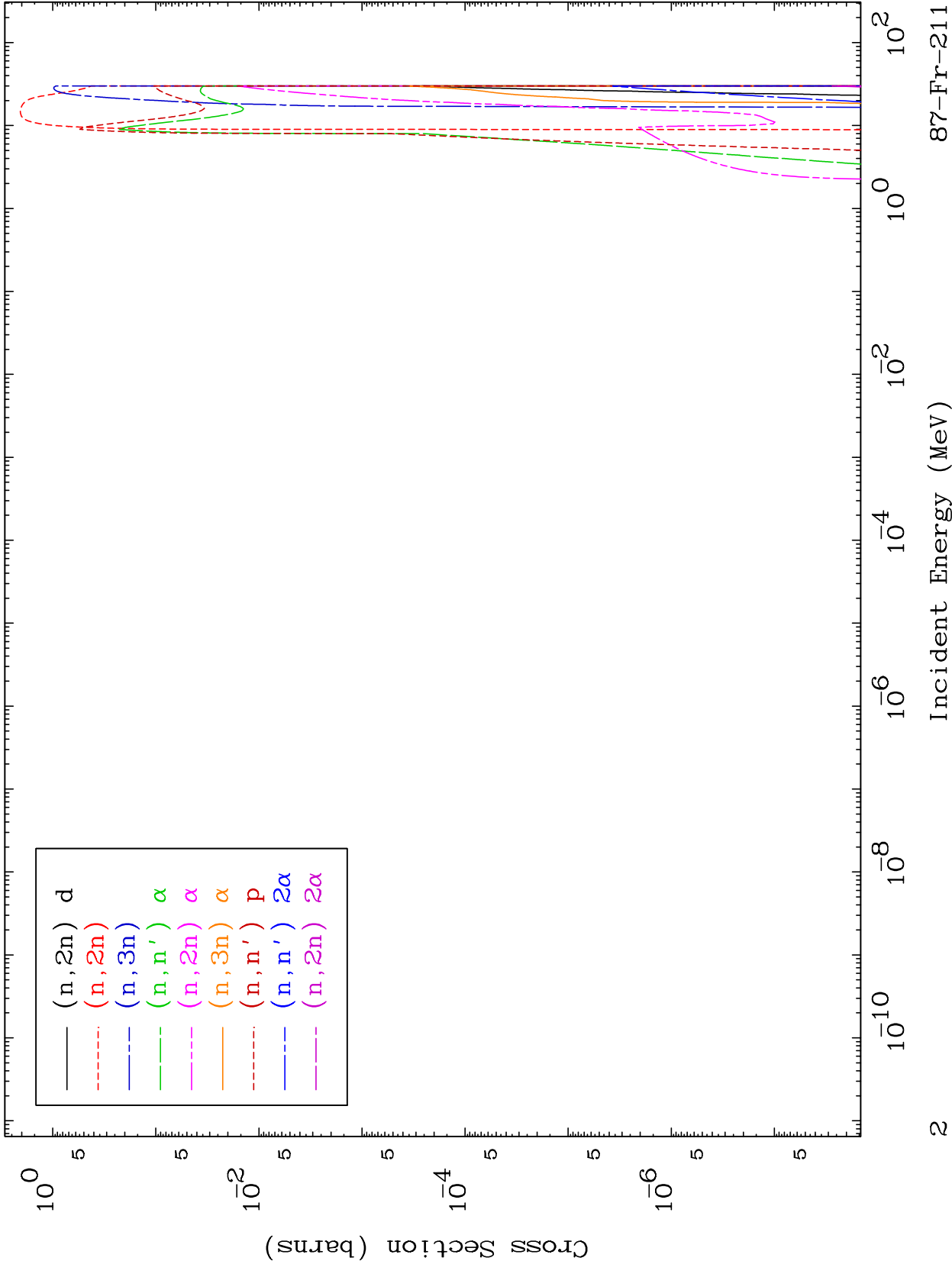
293 Kelvin Cross Sections

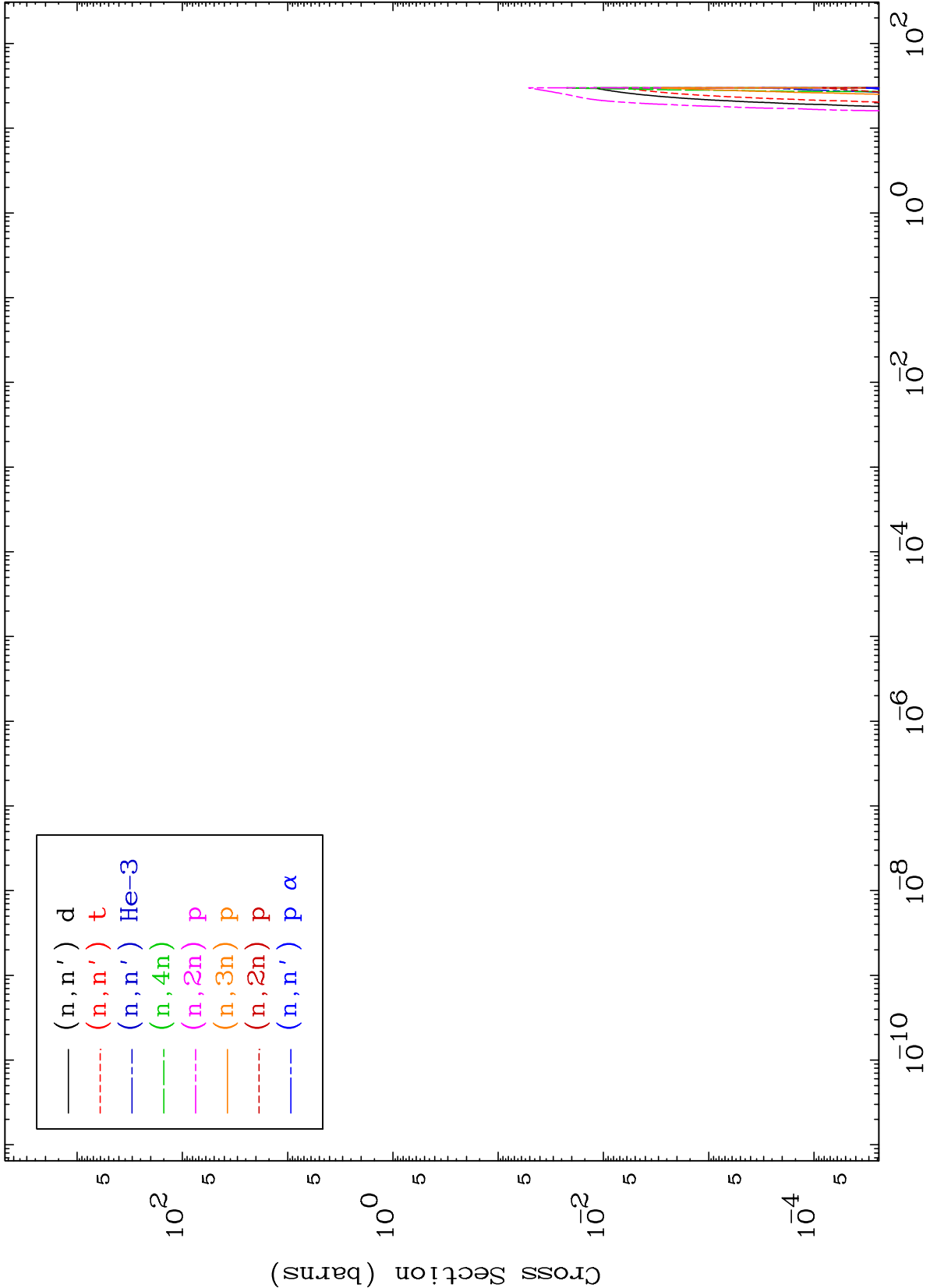


1

Incident Energy (MeV)

87-Fr-211

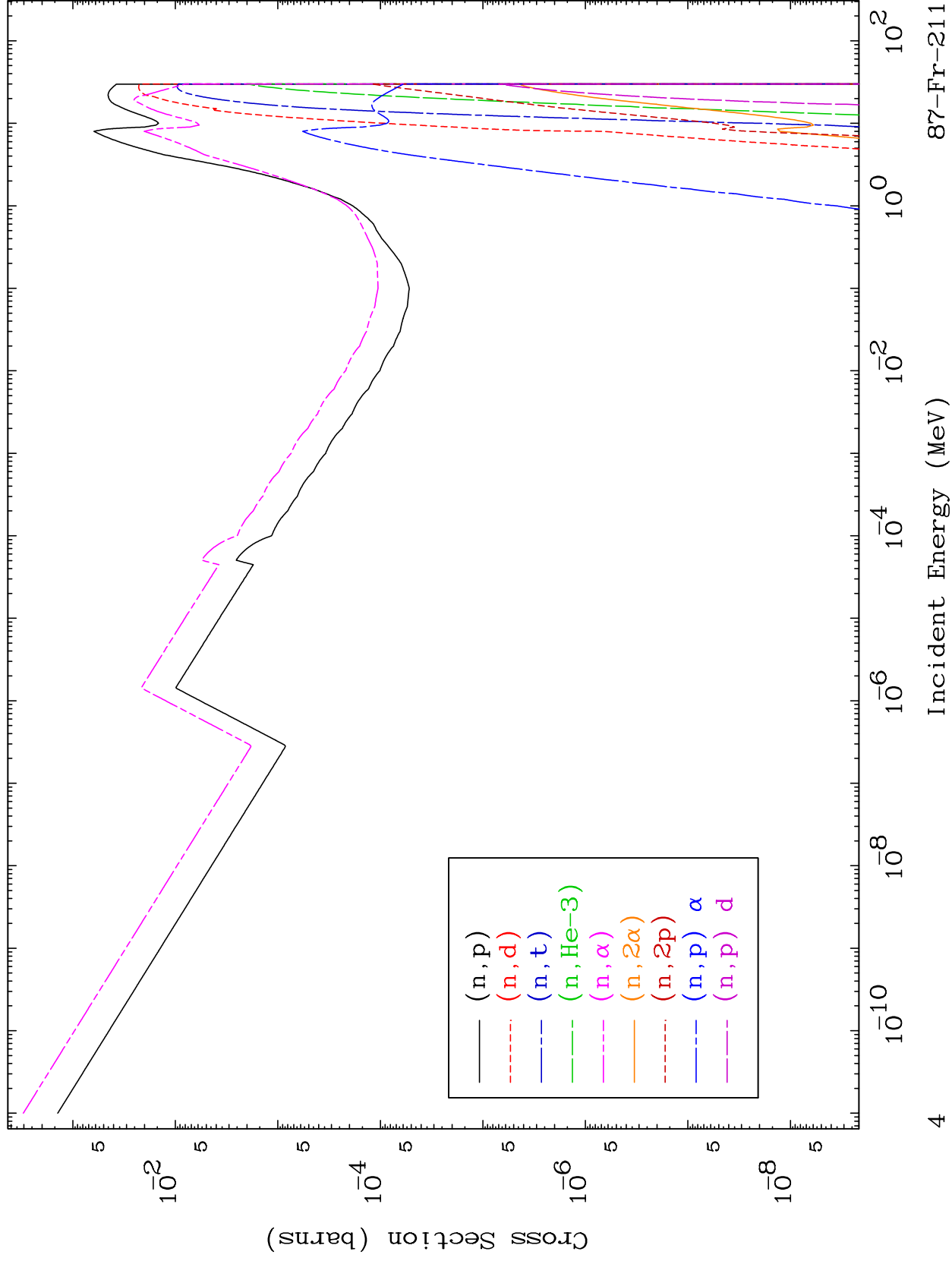


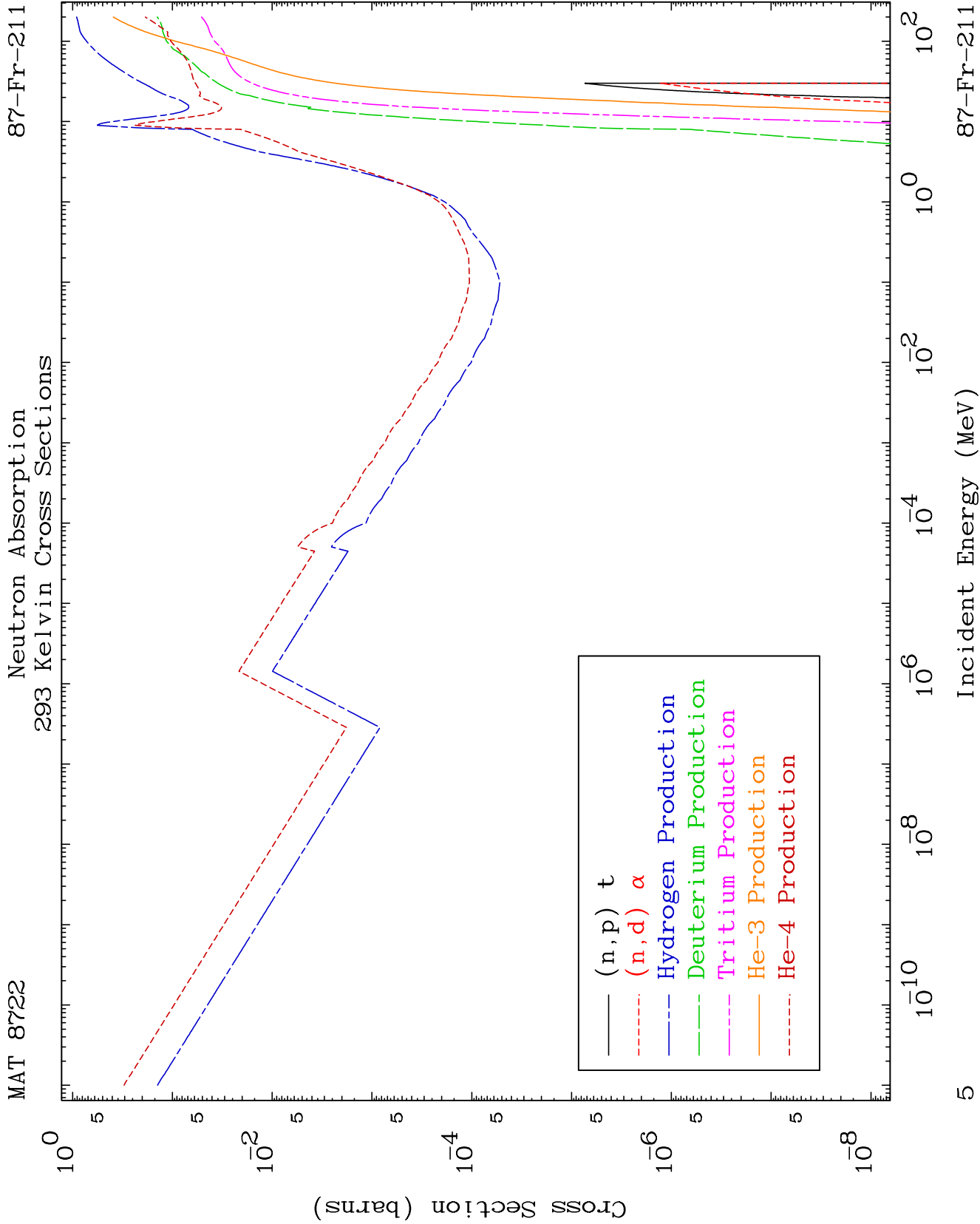


MAT 8722

Neutron Absorption
293 Kelvin Cross Sections

87-Fr-211

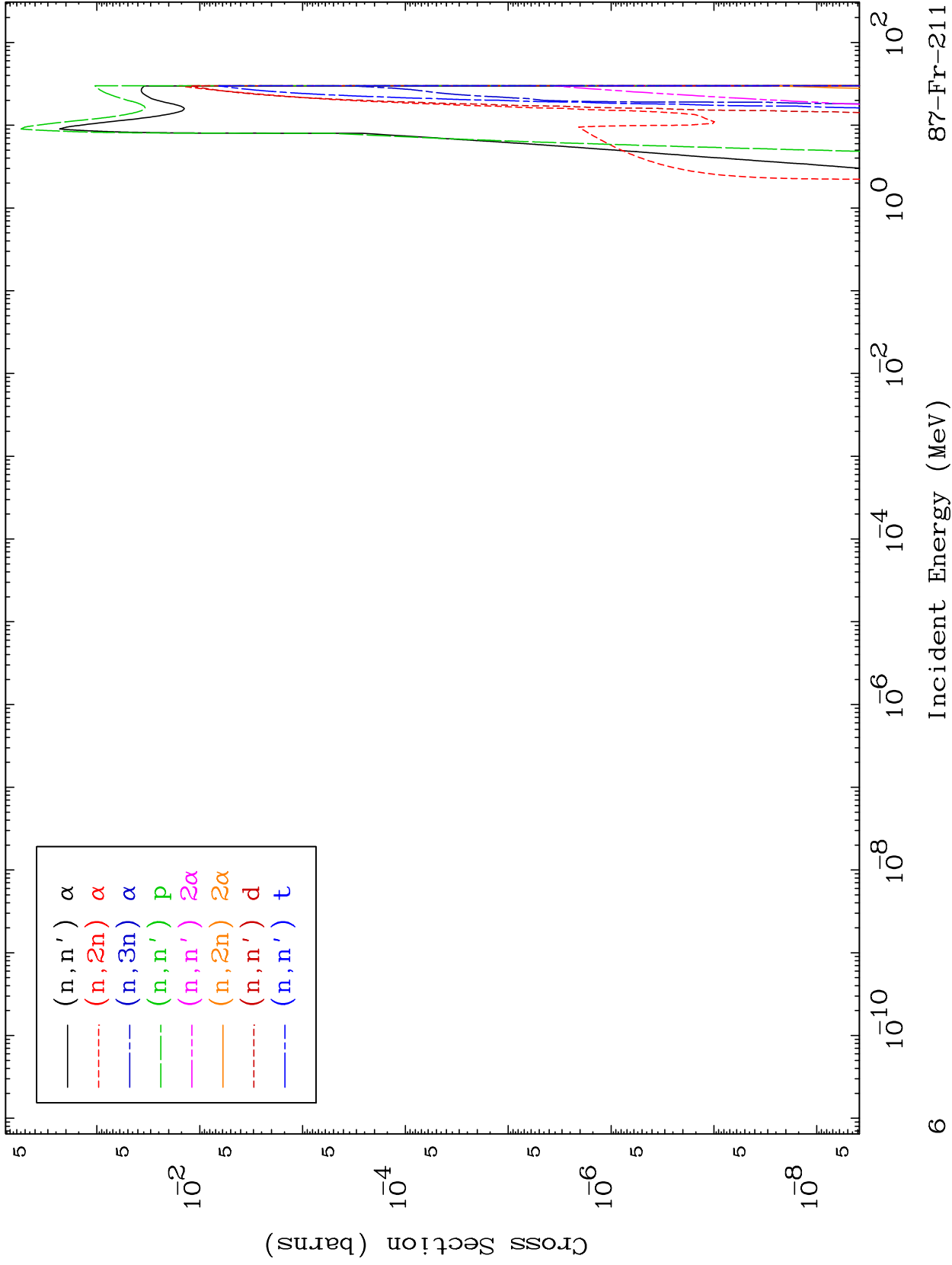




MAT 8722

Charged Particle
293 Kelvin Cross Sections

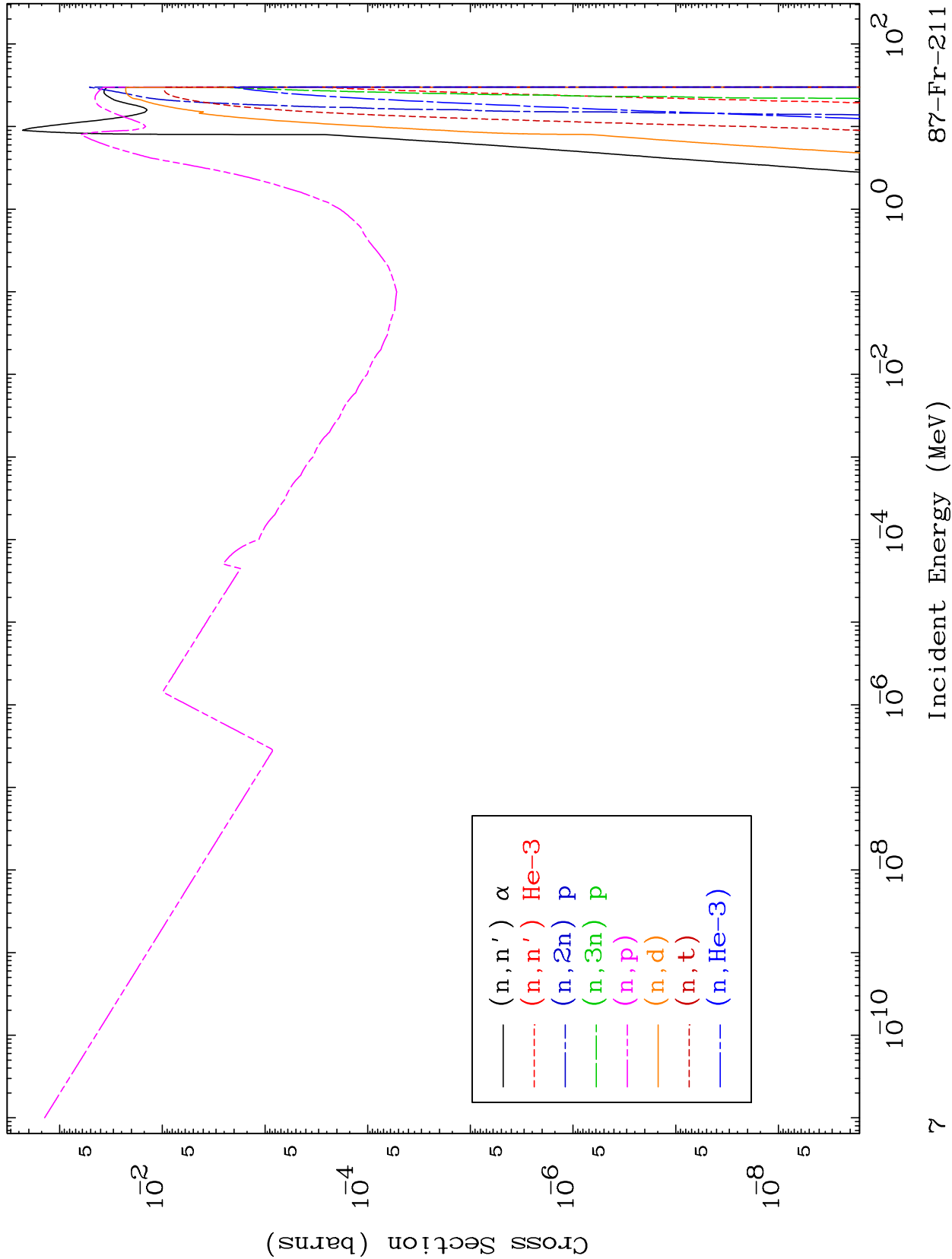
87-Fr-211

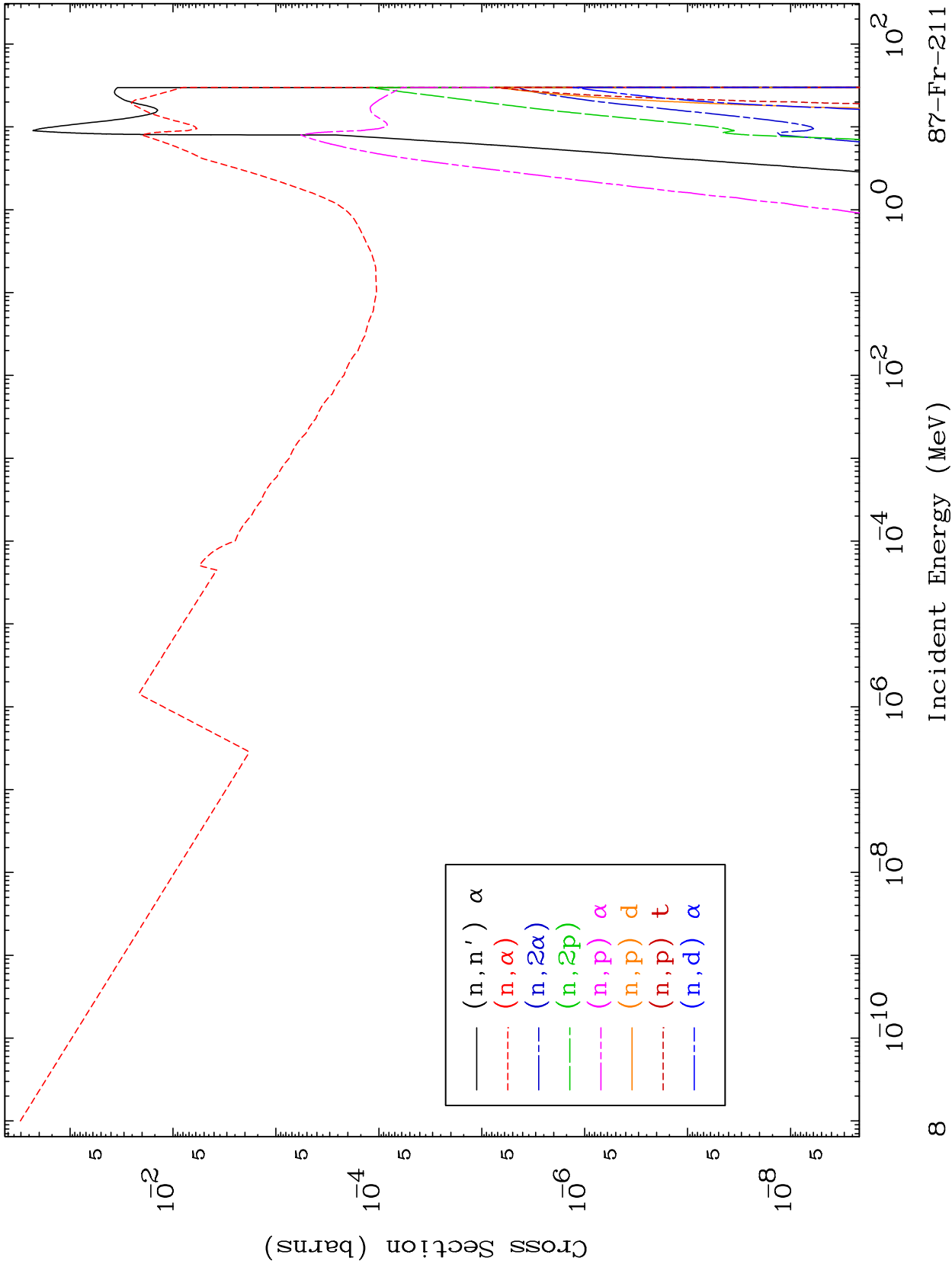


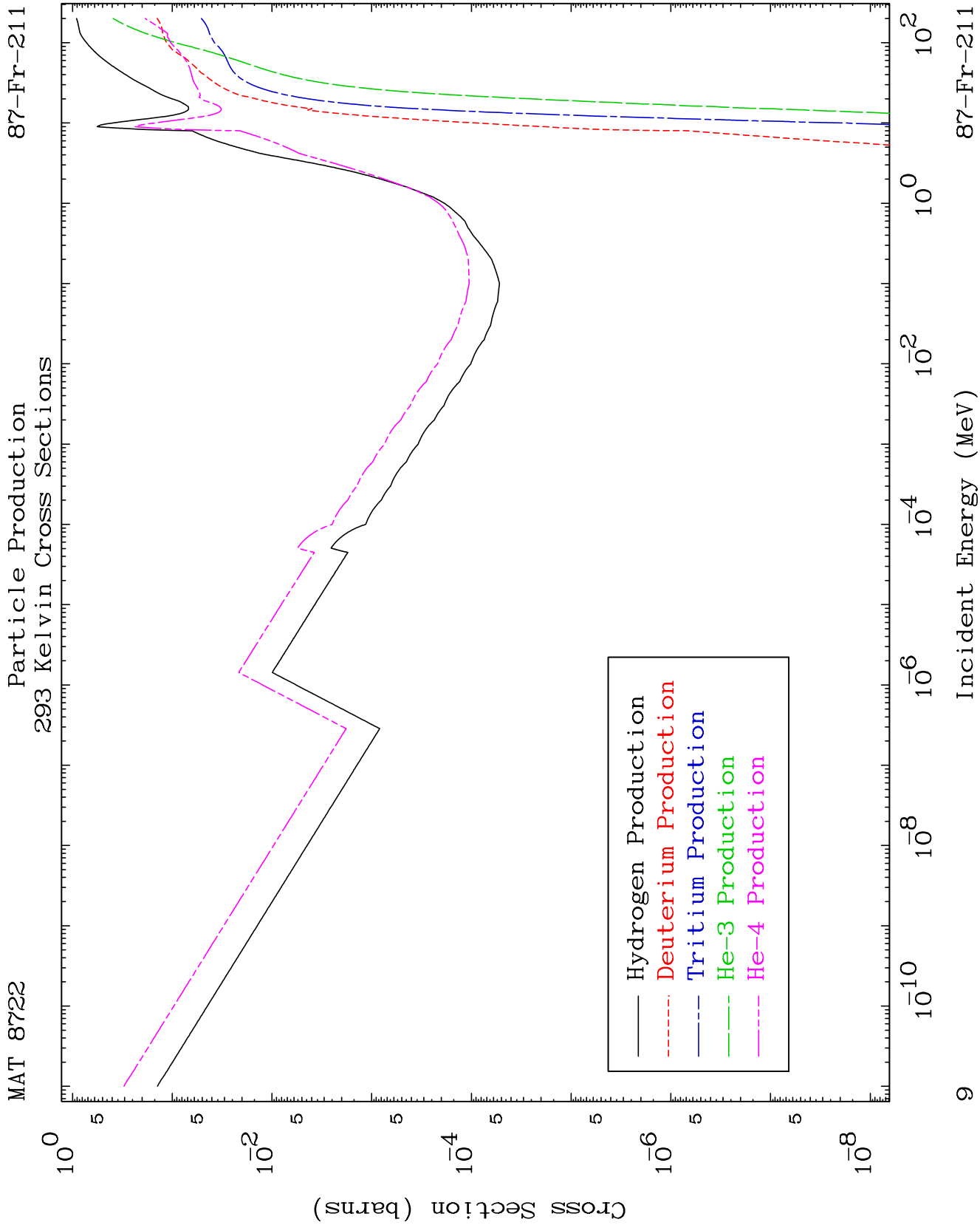
MAT 8722

Charged Particle
293 Kelvin Cross Sections

87-Fr-211





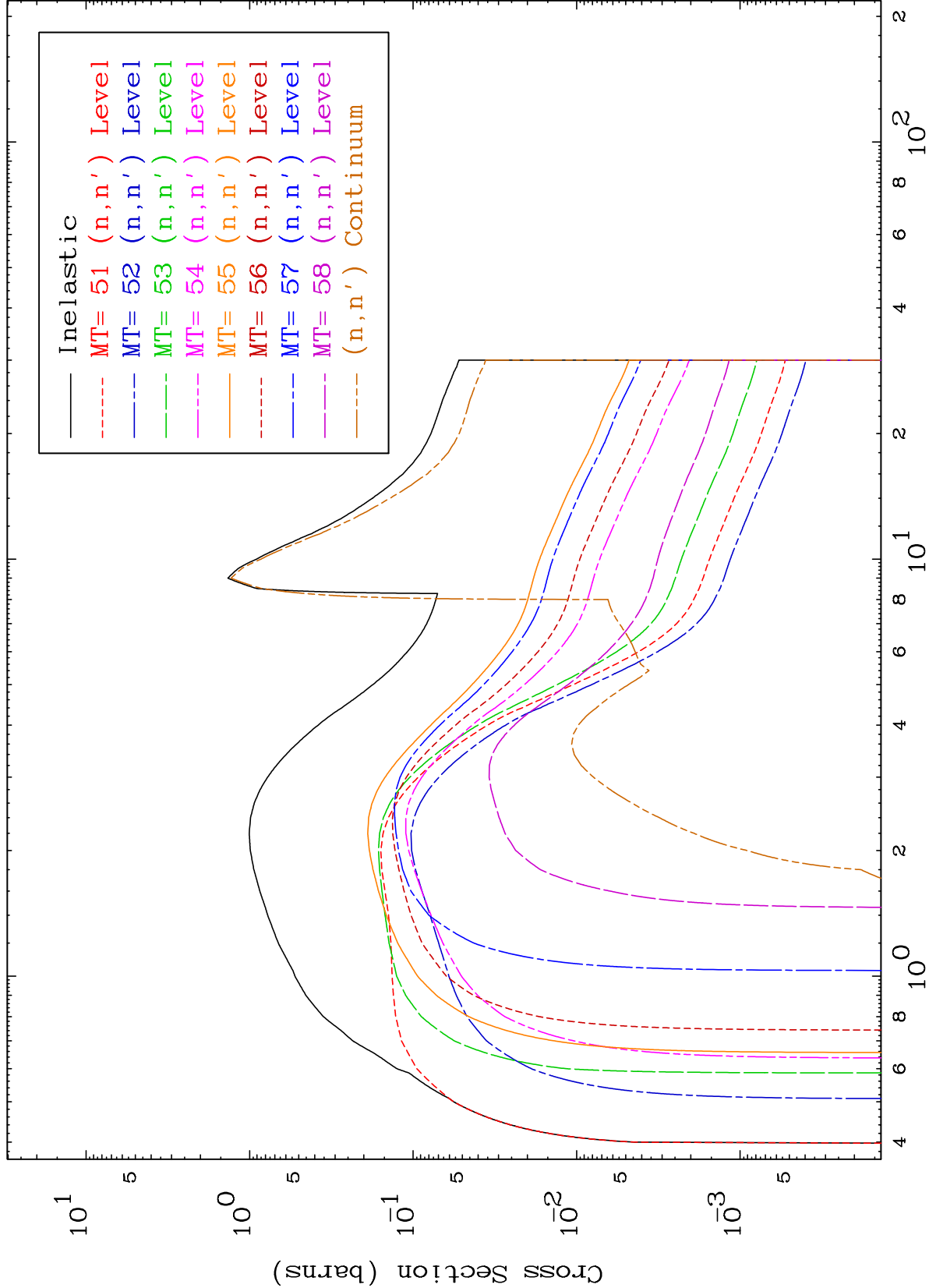


MAT 8722

(n,n') Levels

293 Kelvin Cross Sections

87-Fr-211



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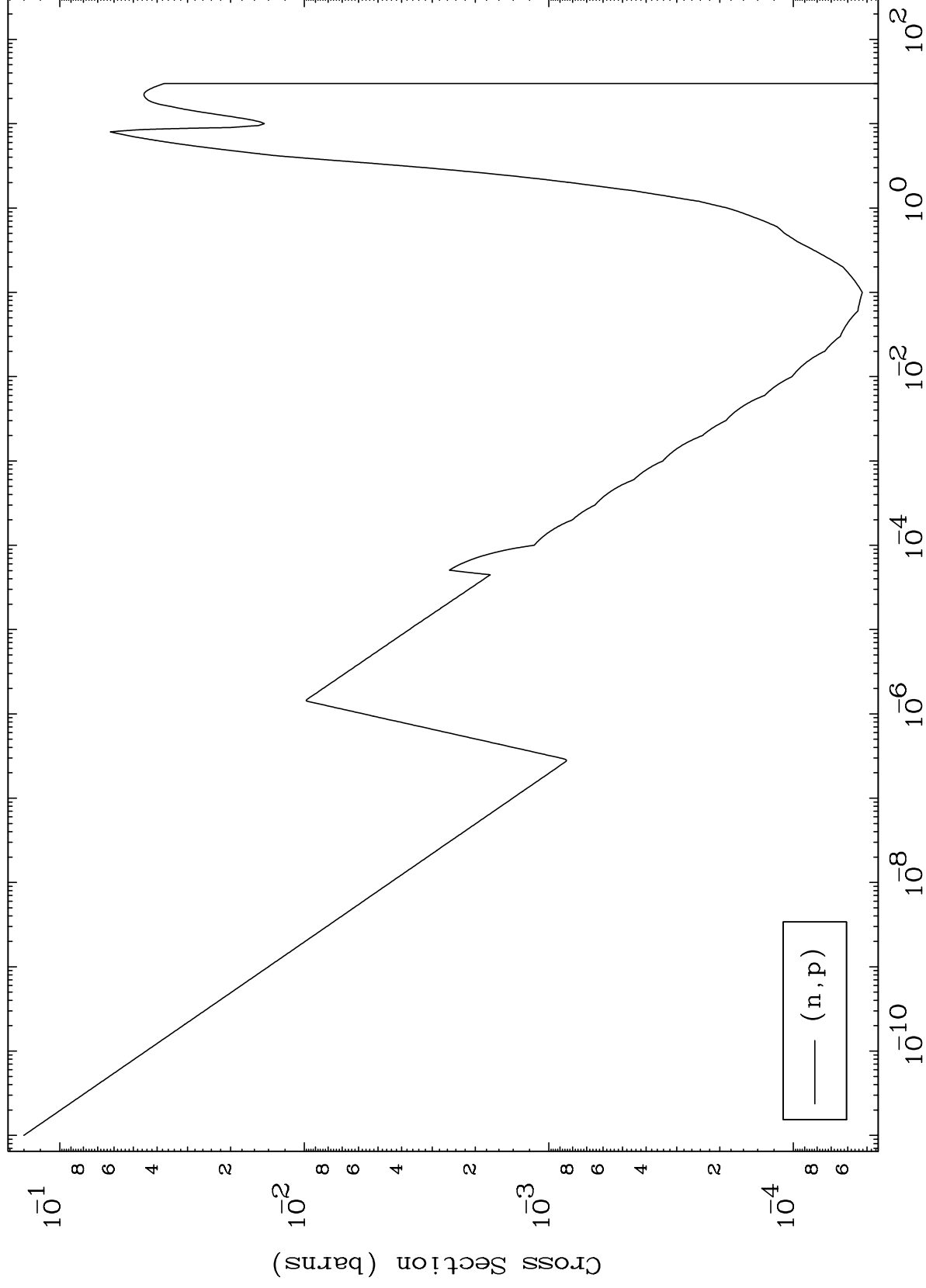
Incident Energy (MeV)

87-Fr-211

MAT 8722

87-Fr-211

(n,p) Levels
293 Kelvin Cross Sections



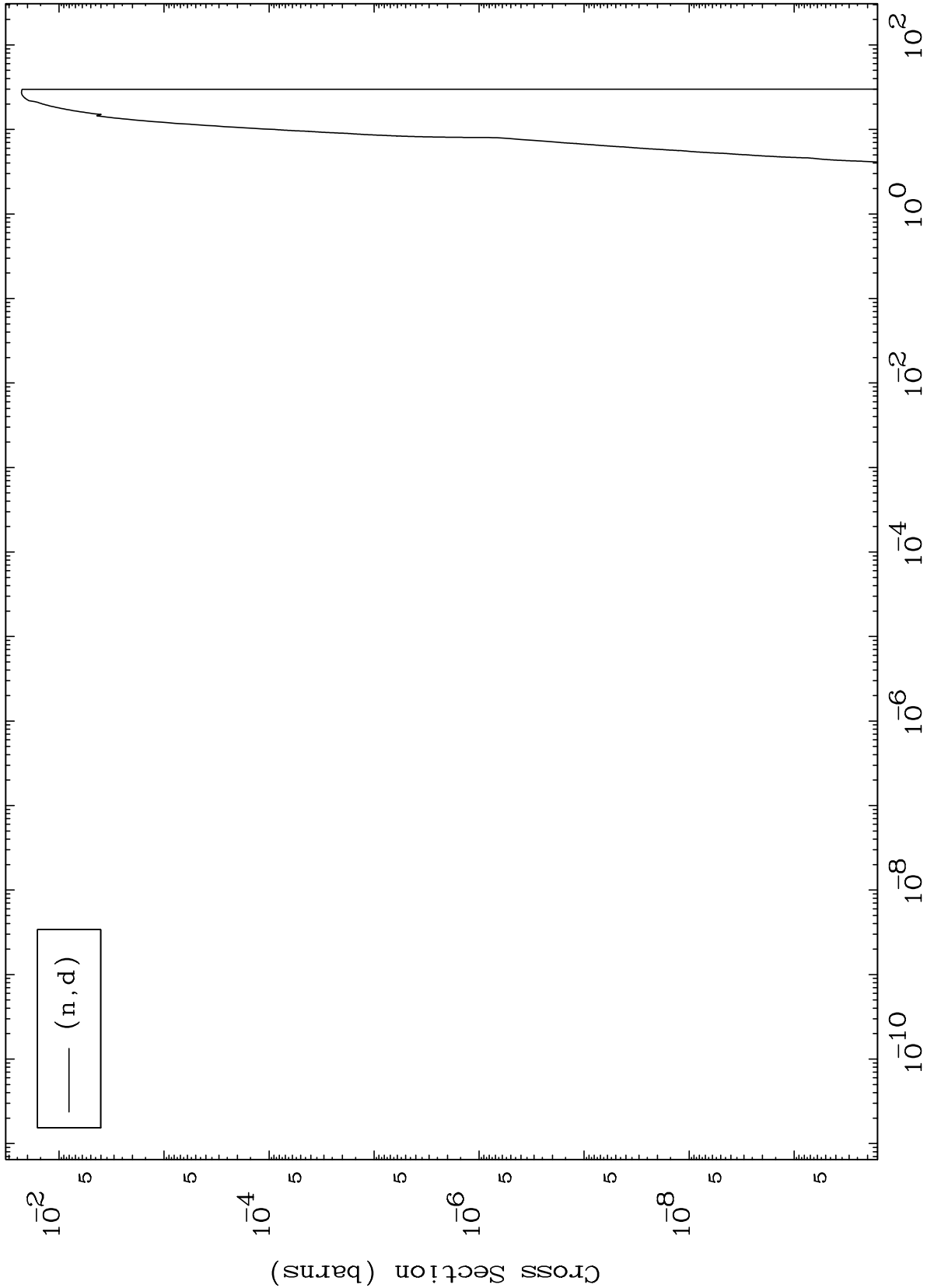
11

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MAT 8722

(n,d) Levels
293 Kelvin Cross Sections

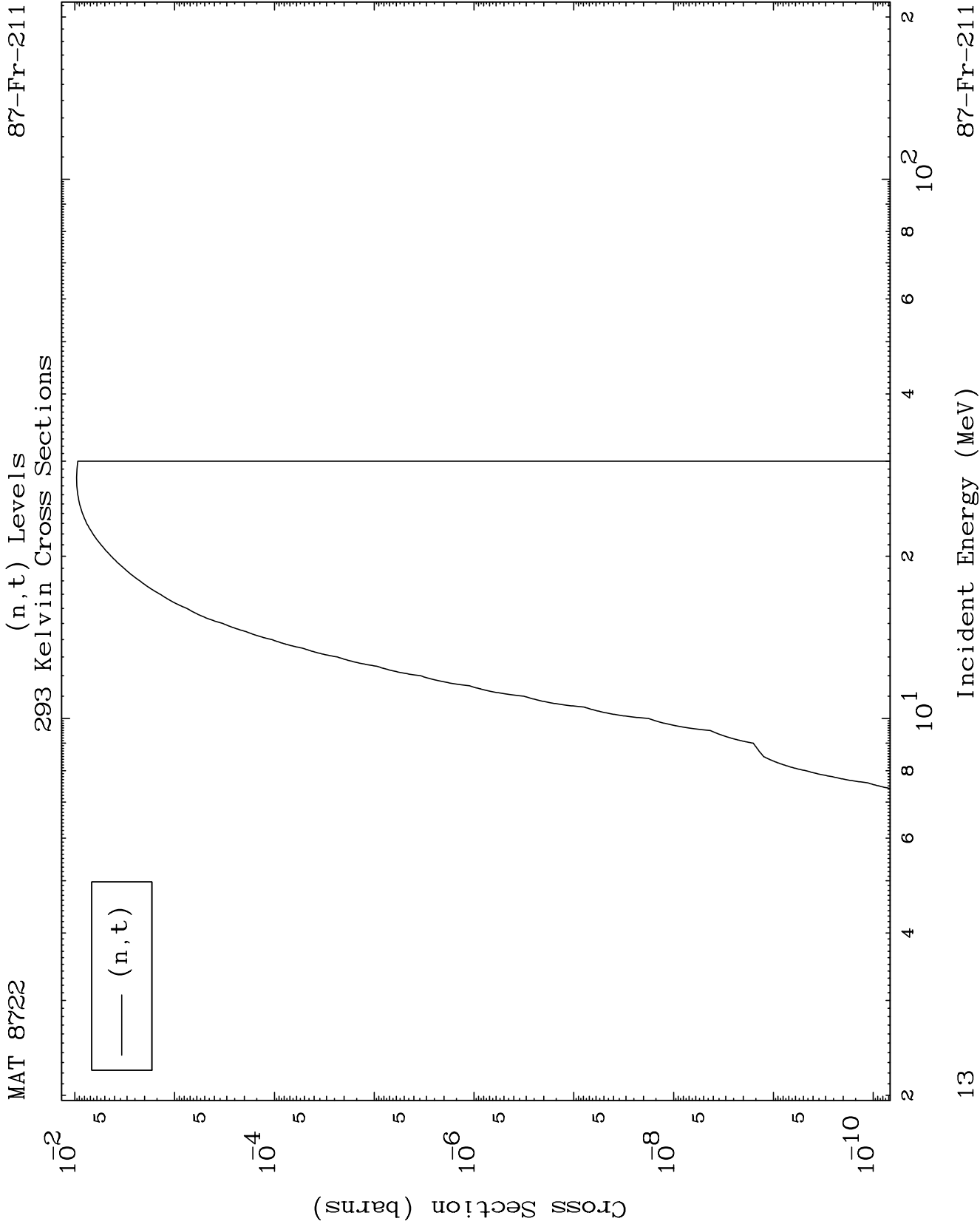
87-Fr-211

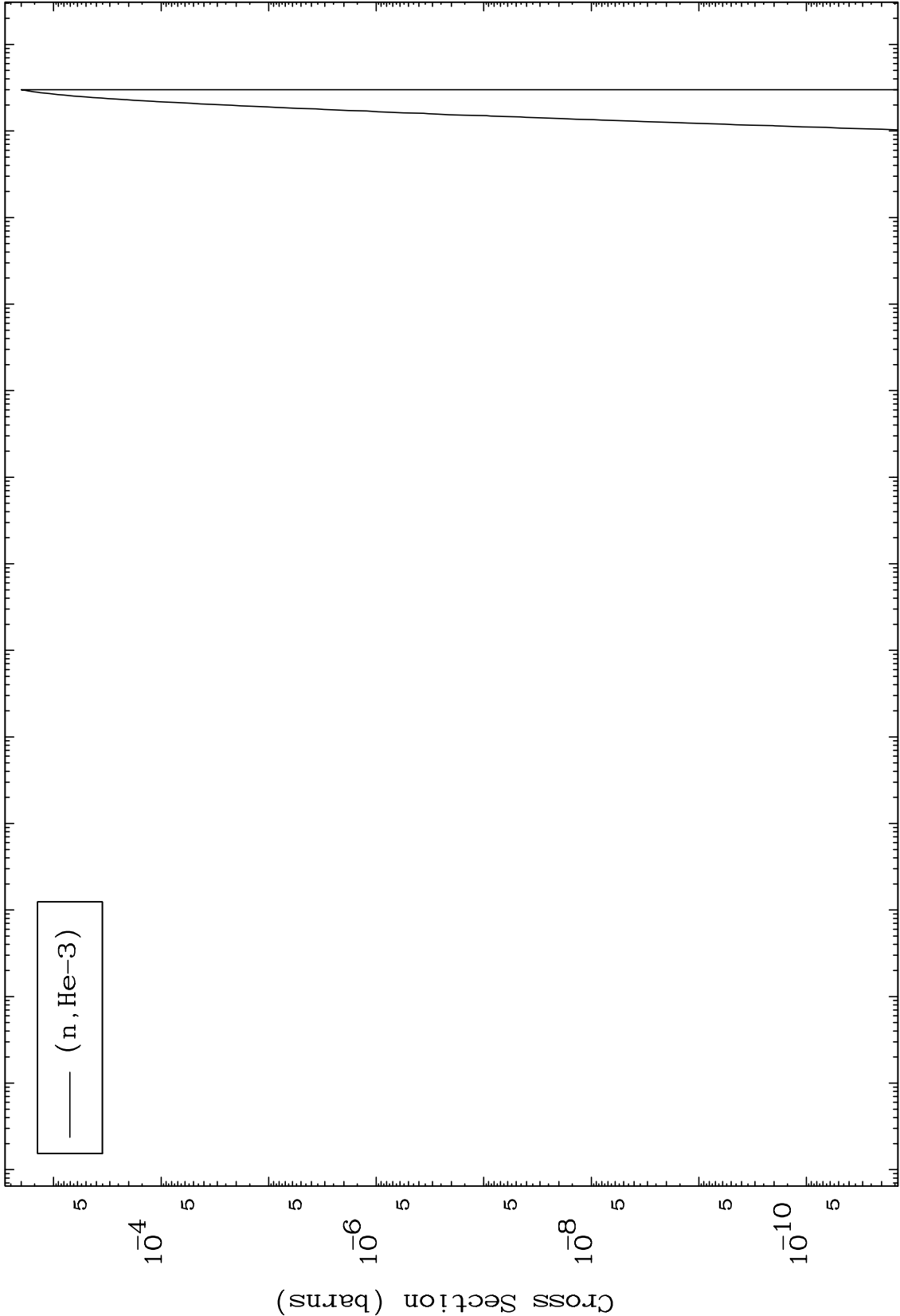


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Incident Energy (MeV)

87-Fr-211

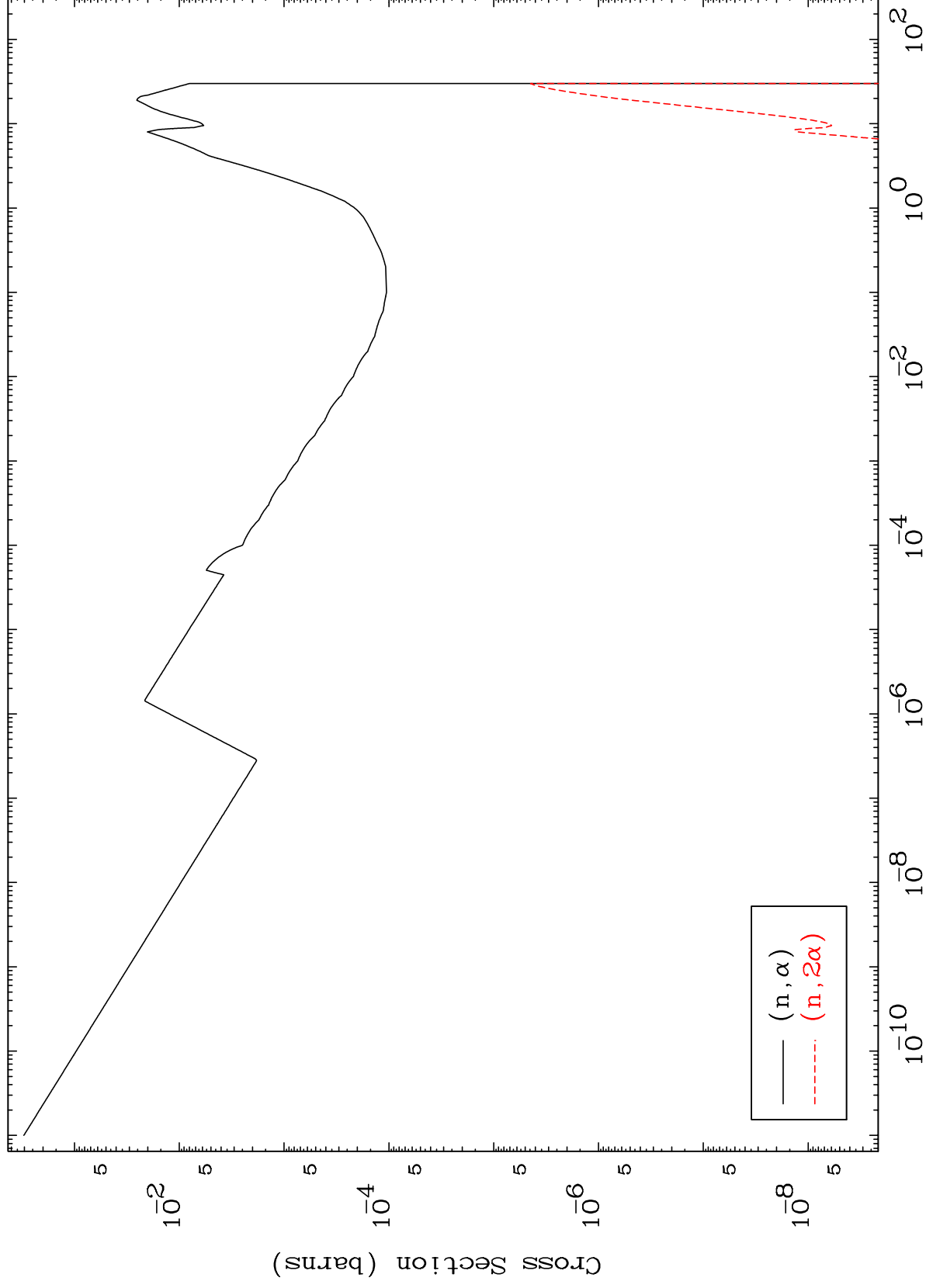




MAT 8722

87-Fr-211

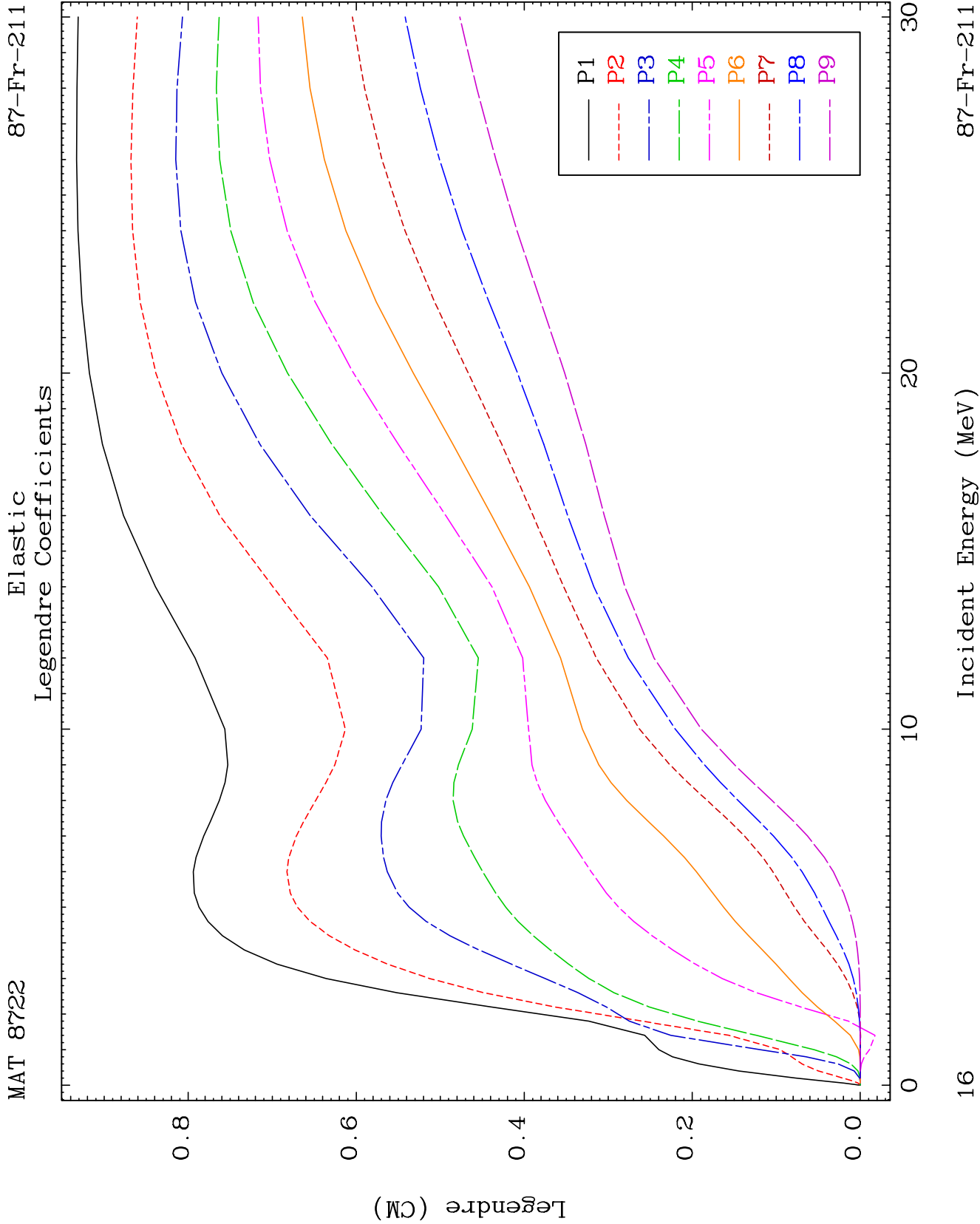
(n, α) Levels
293 Kelvin Cross Sections

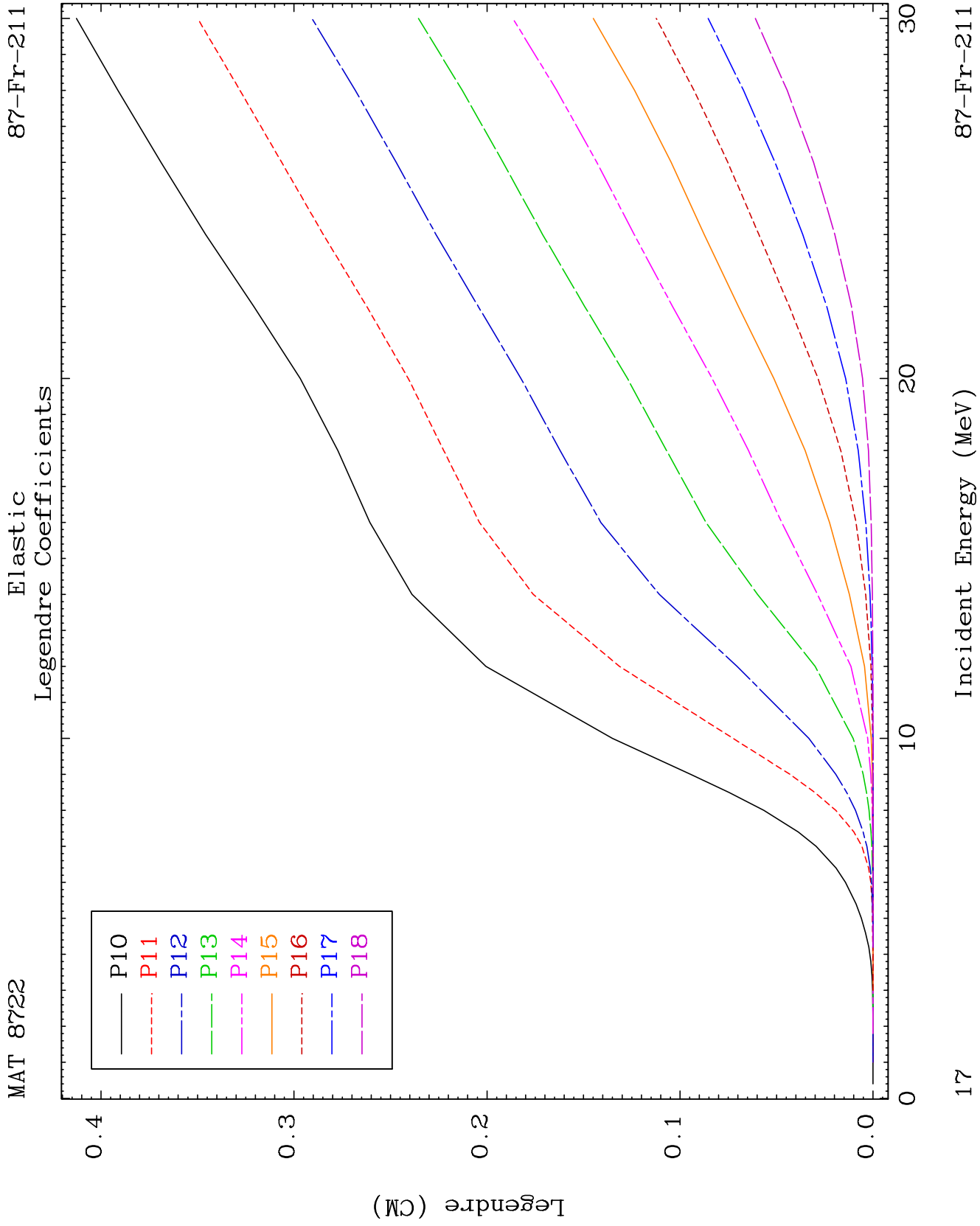


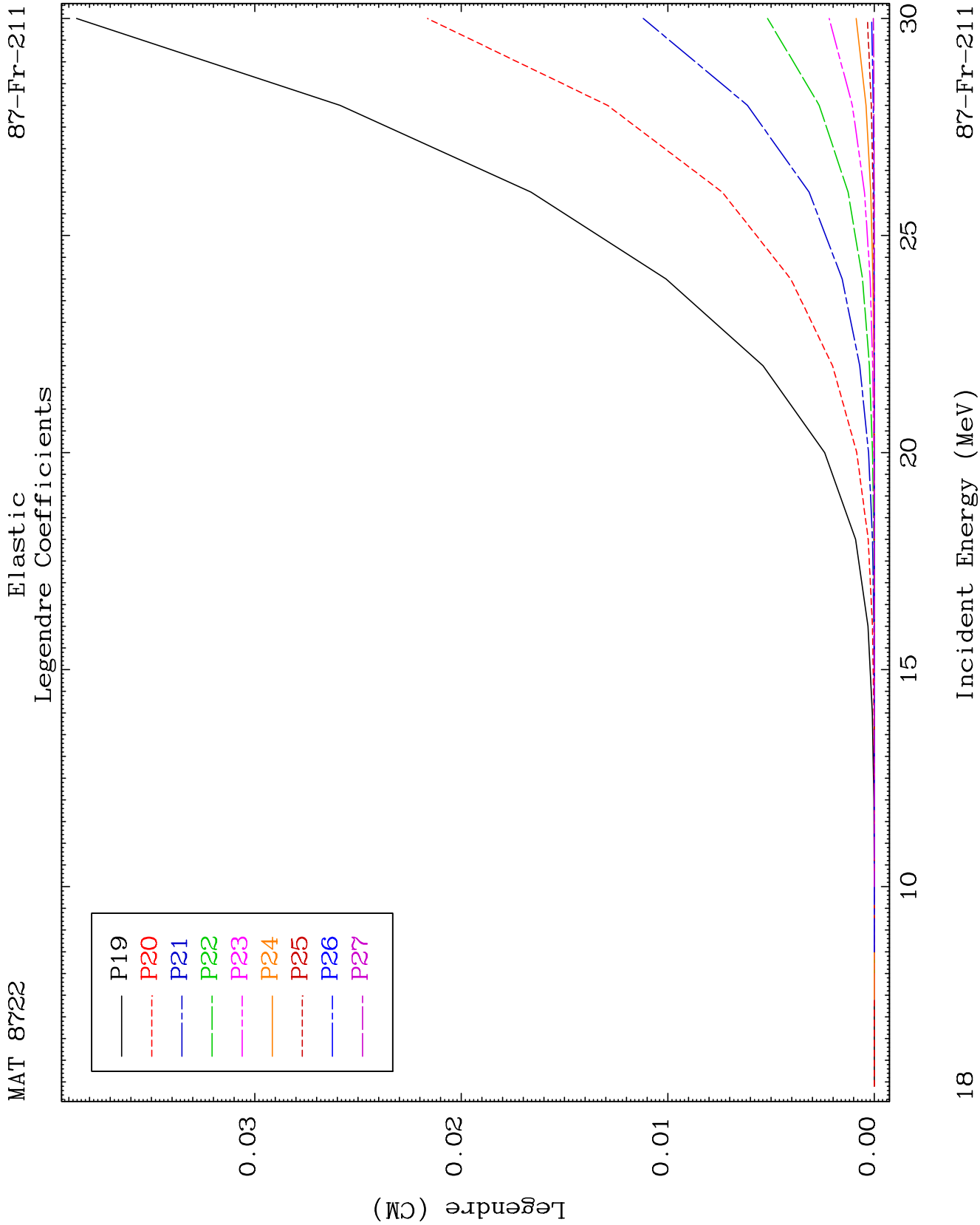
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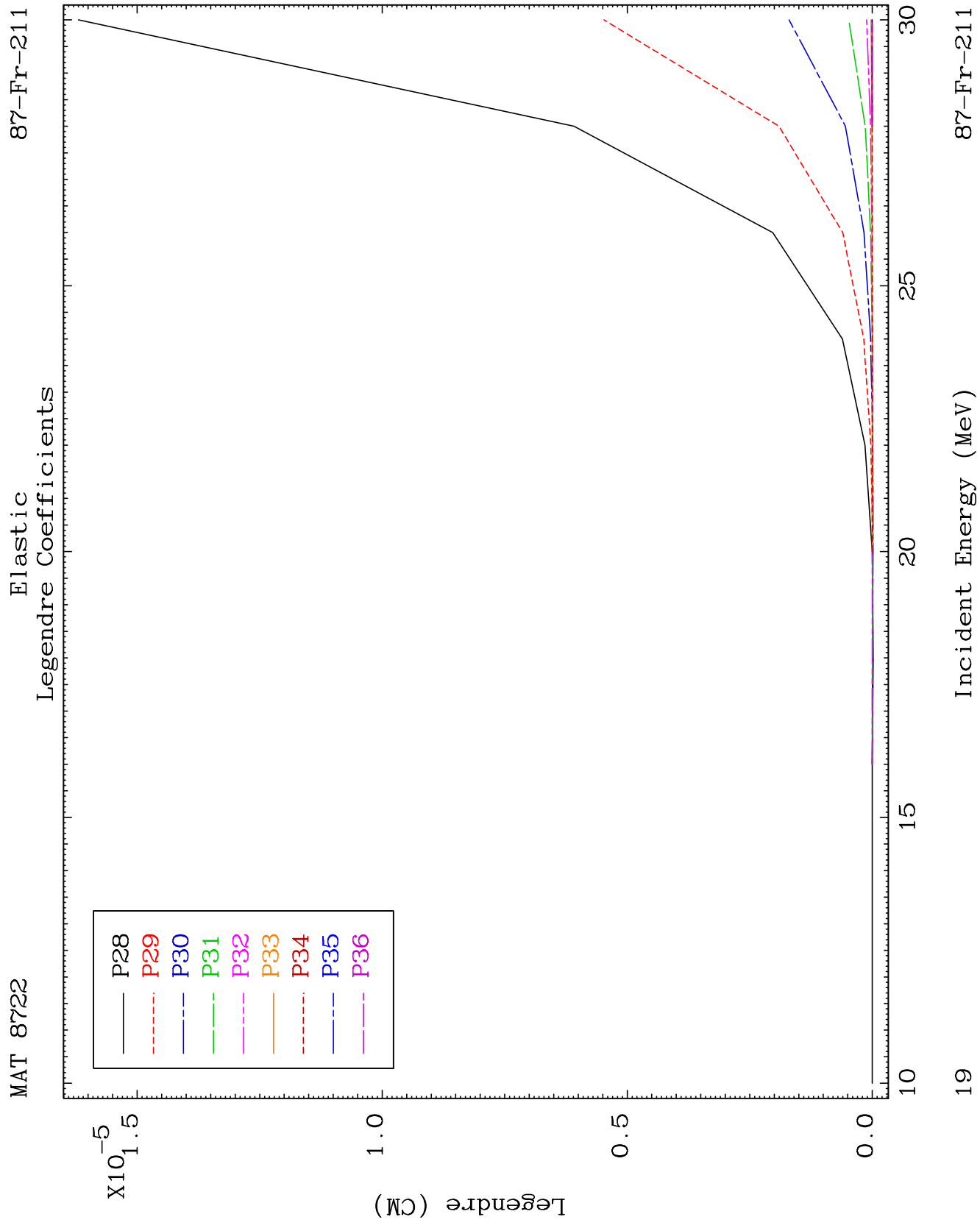
Incident Energy (MeV)

15





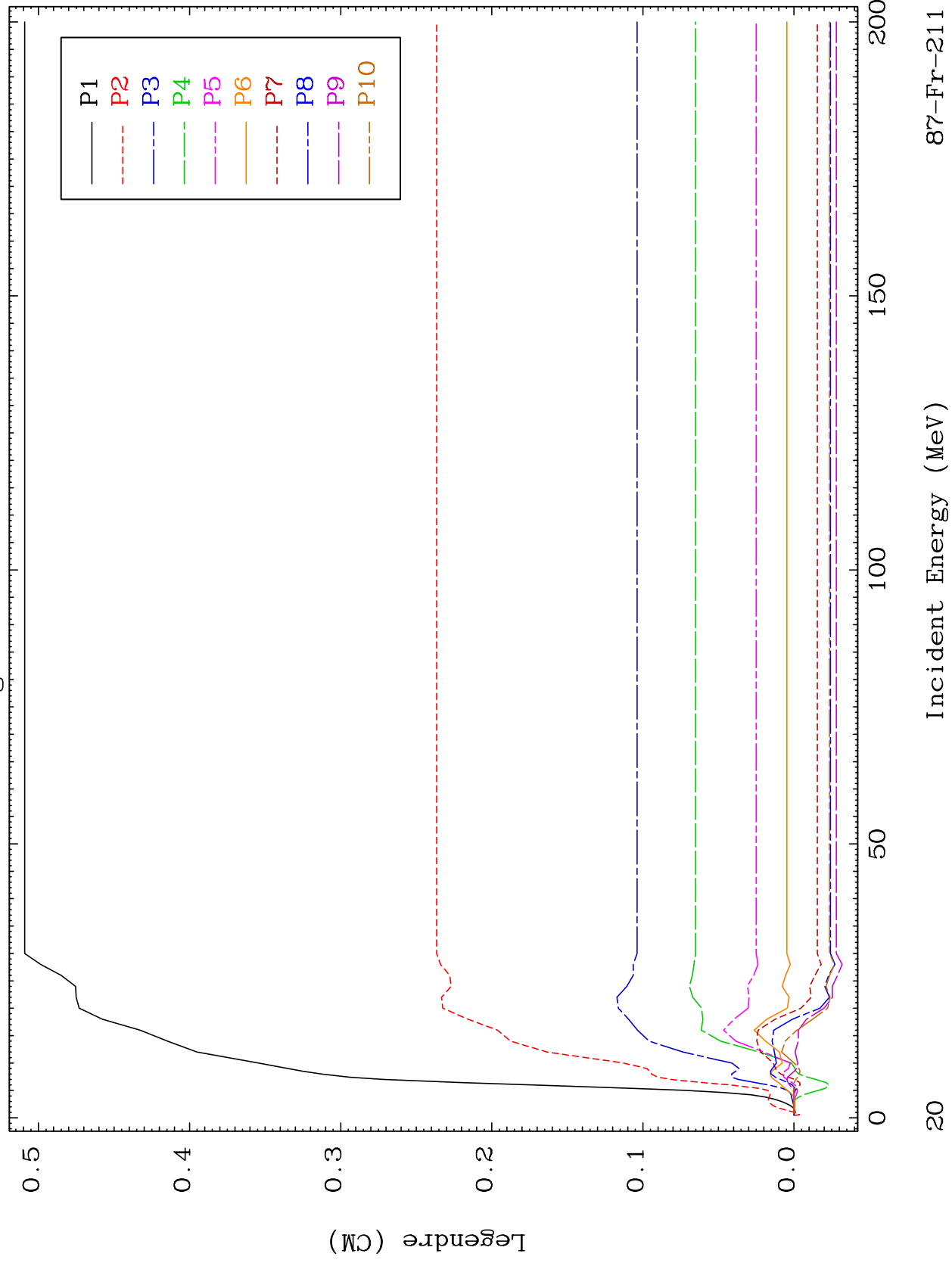




MAT 8722

MT= 51 (n, n') Level
Legendre Coefficients

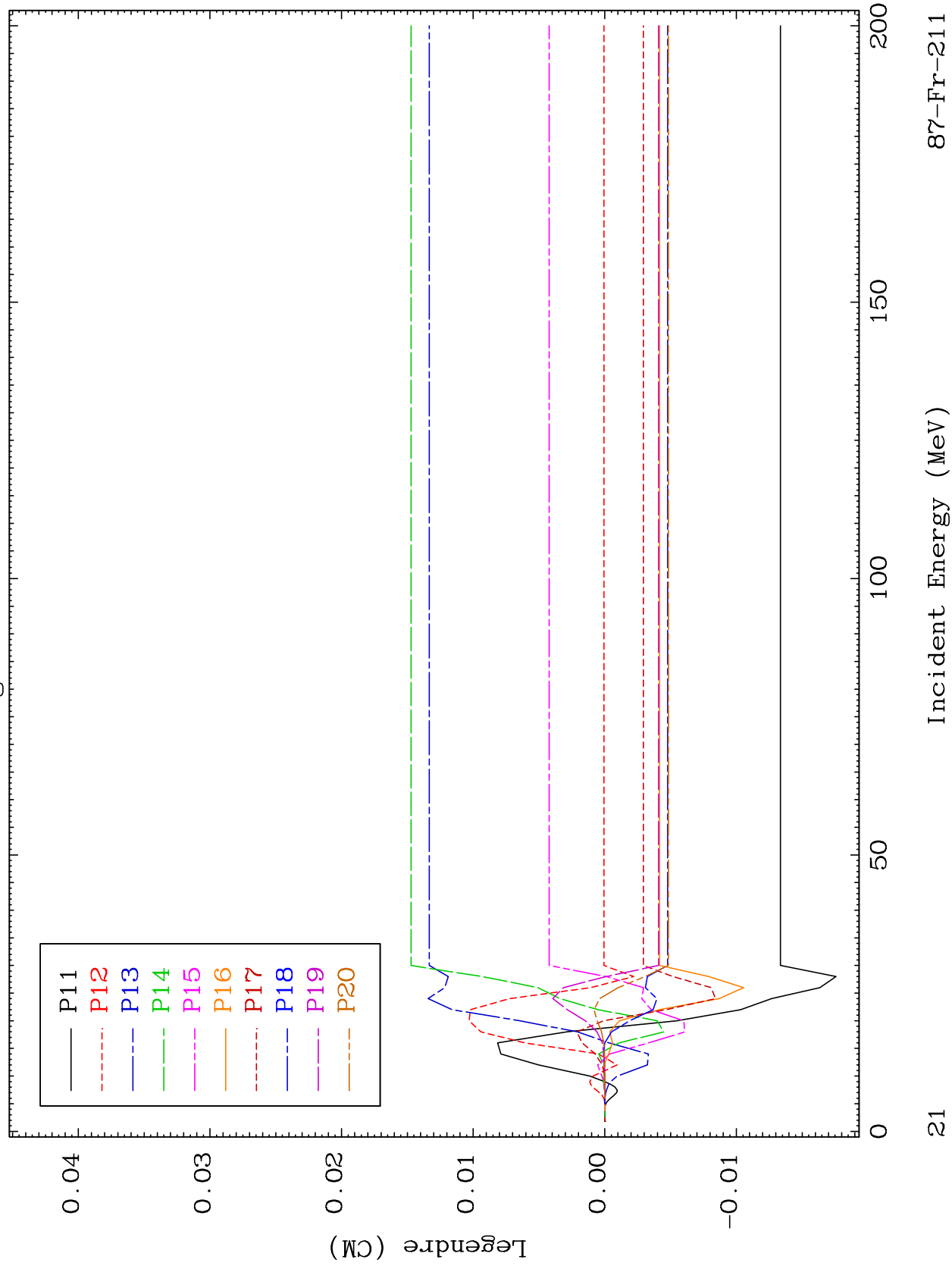
87-Fr-211



MAT 8722

MT= 51 (n, n') Level
Legendre Coefficients

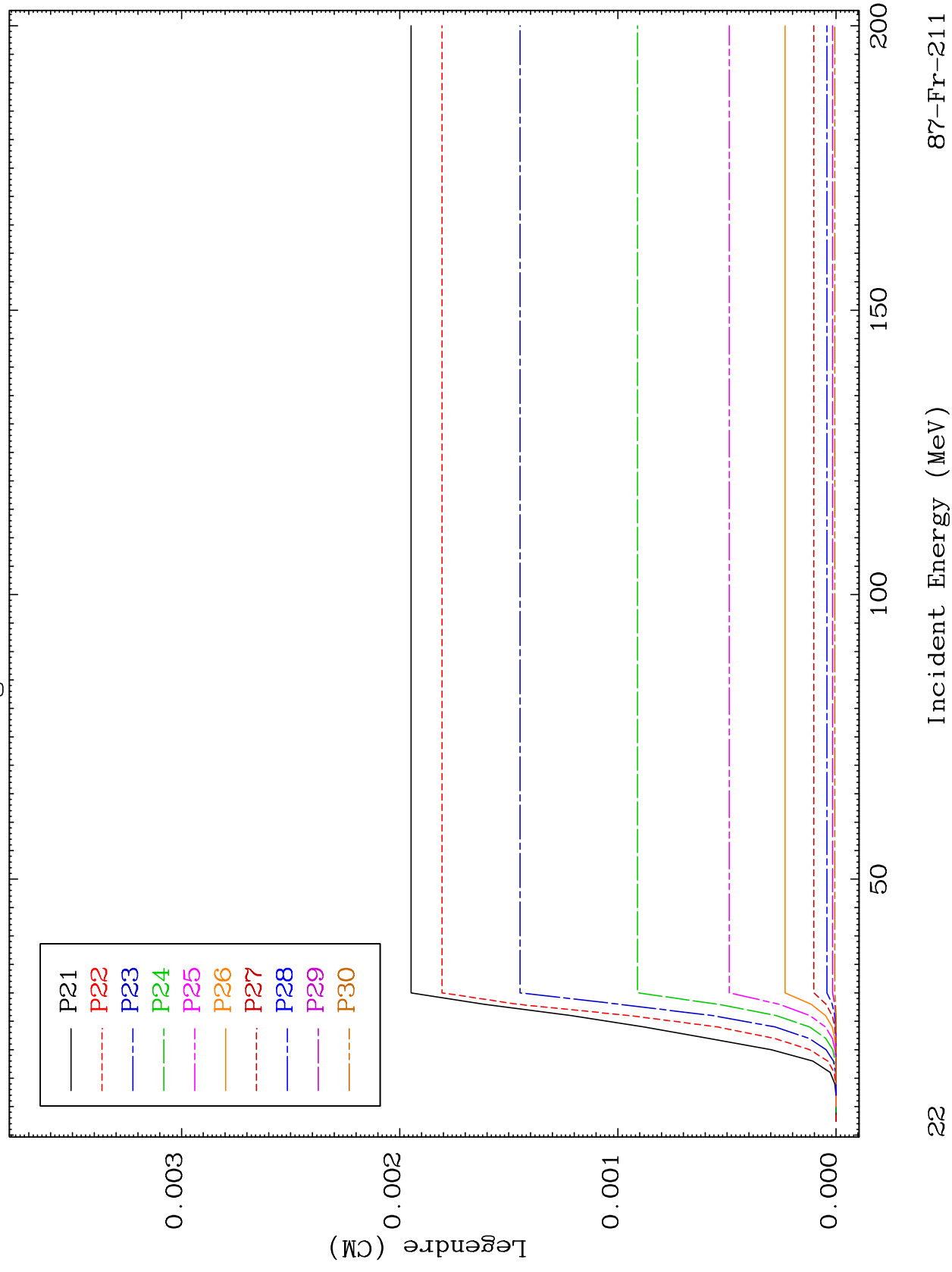
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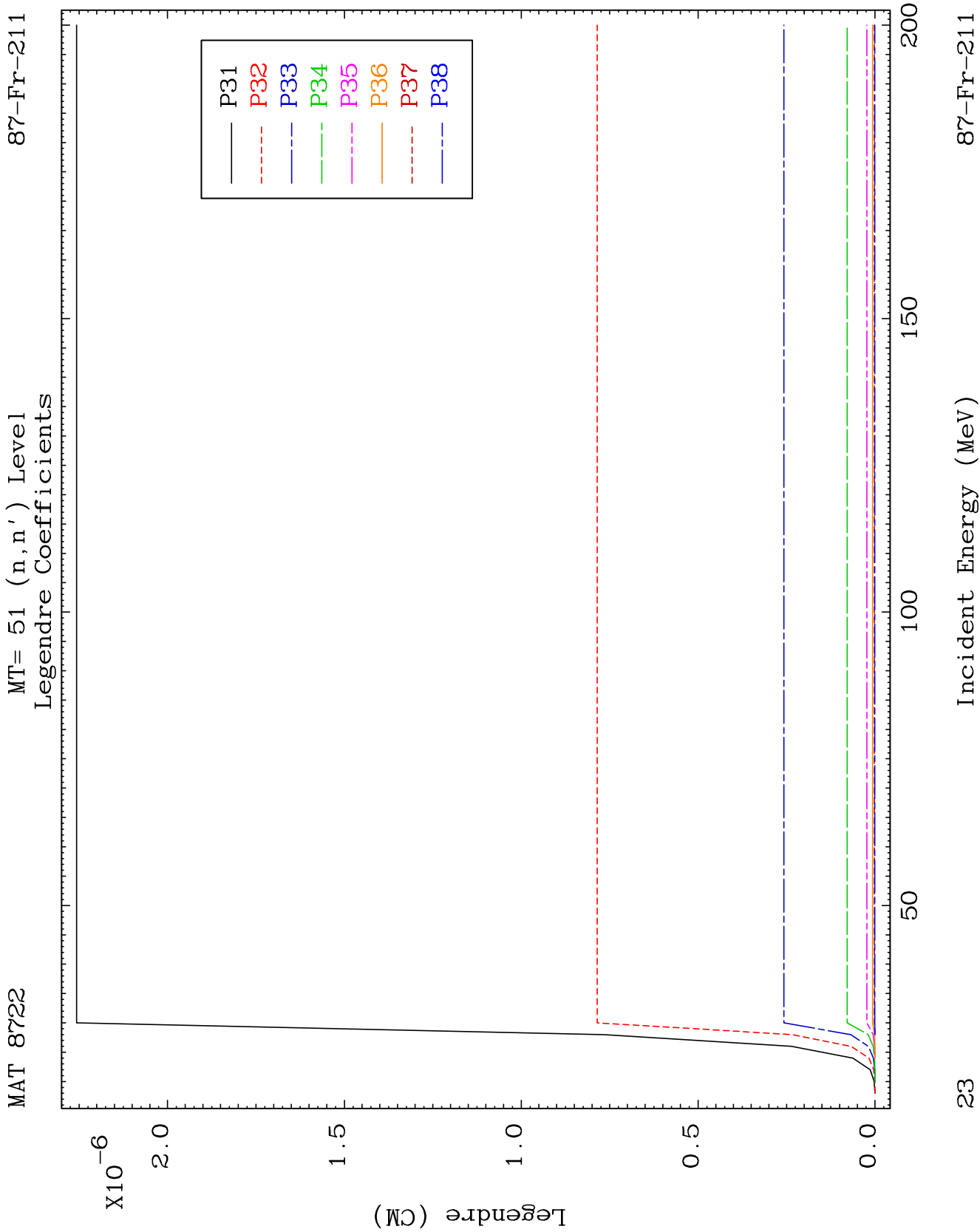


MAT 8722

MT= 51 (n, n') Level
Legendre Coefficients

87-Fr-211

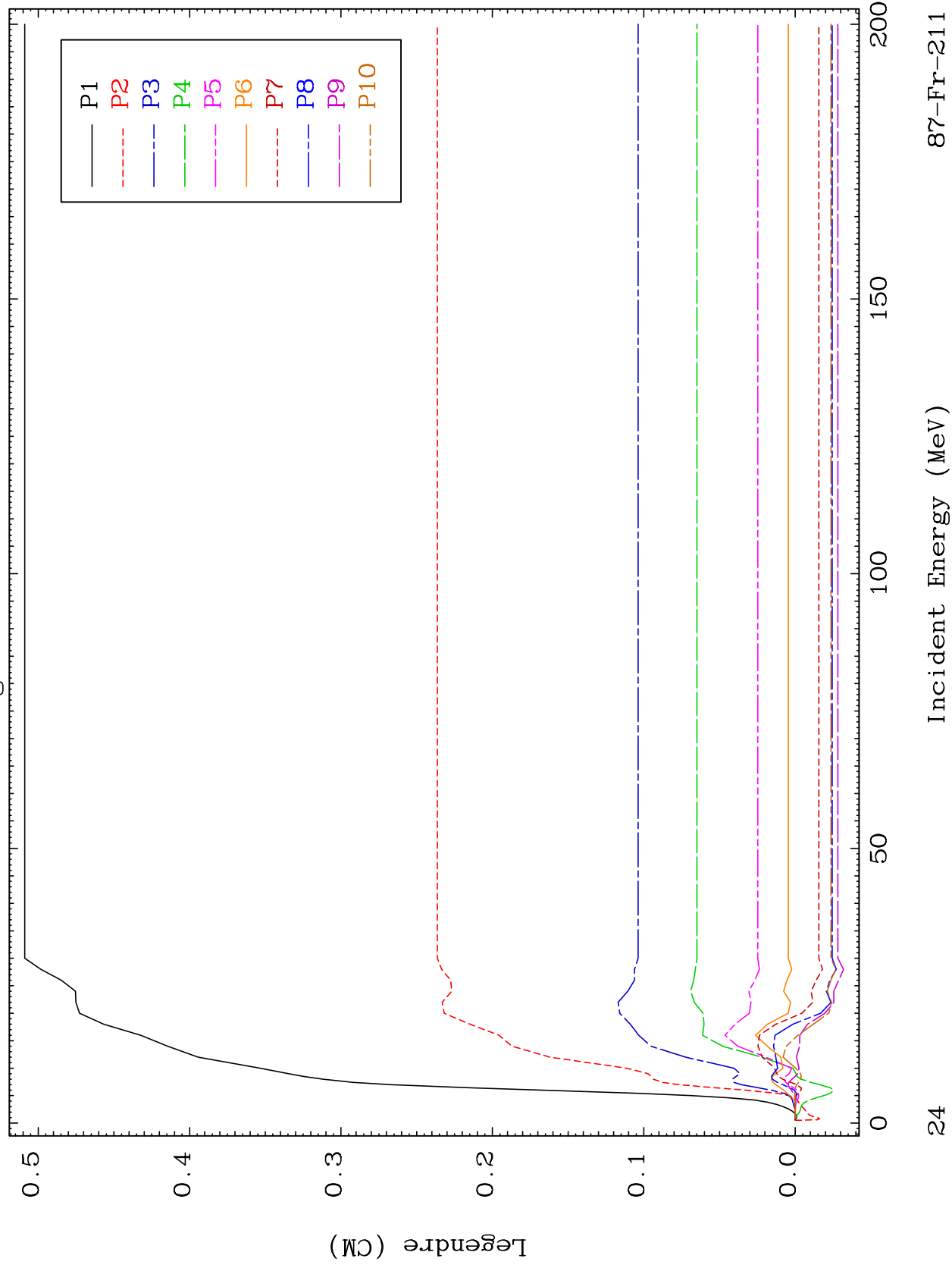




MAT 8722

MT= 52 (n, n') Level
Legendre Coefficients

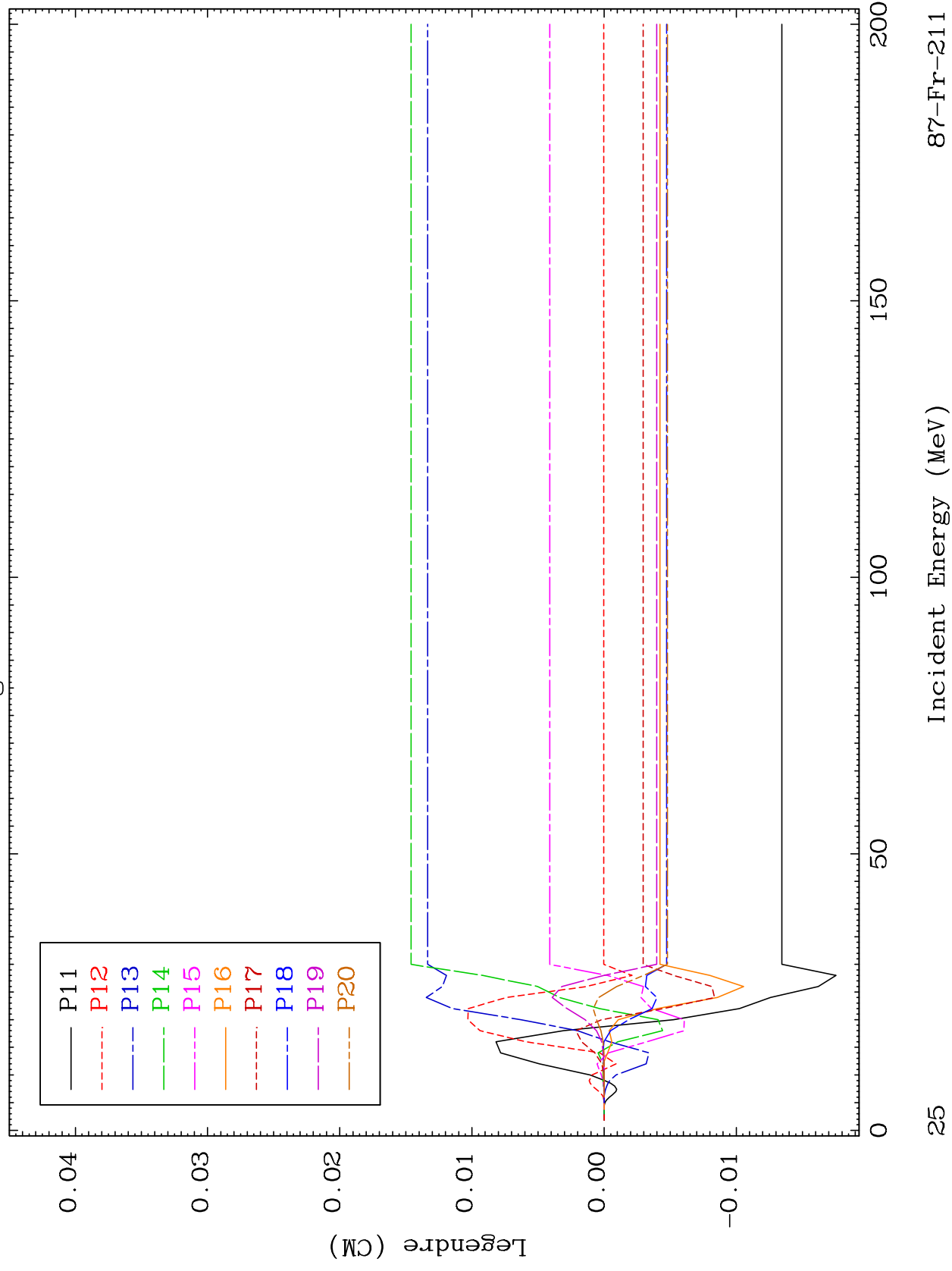
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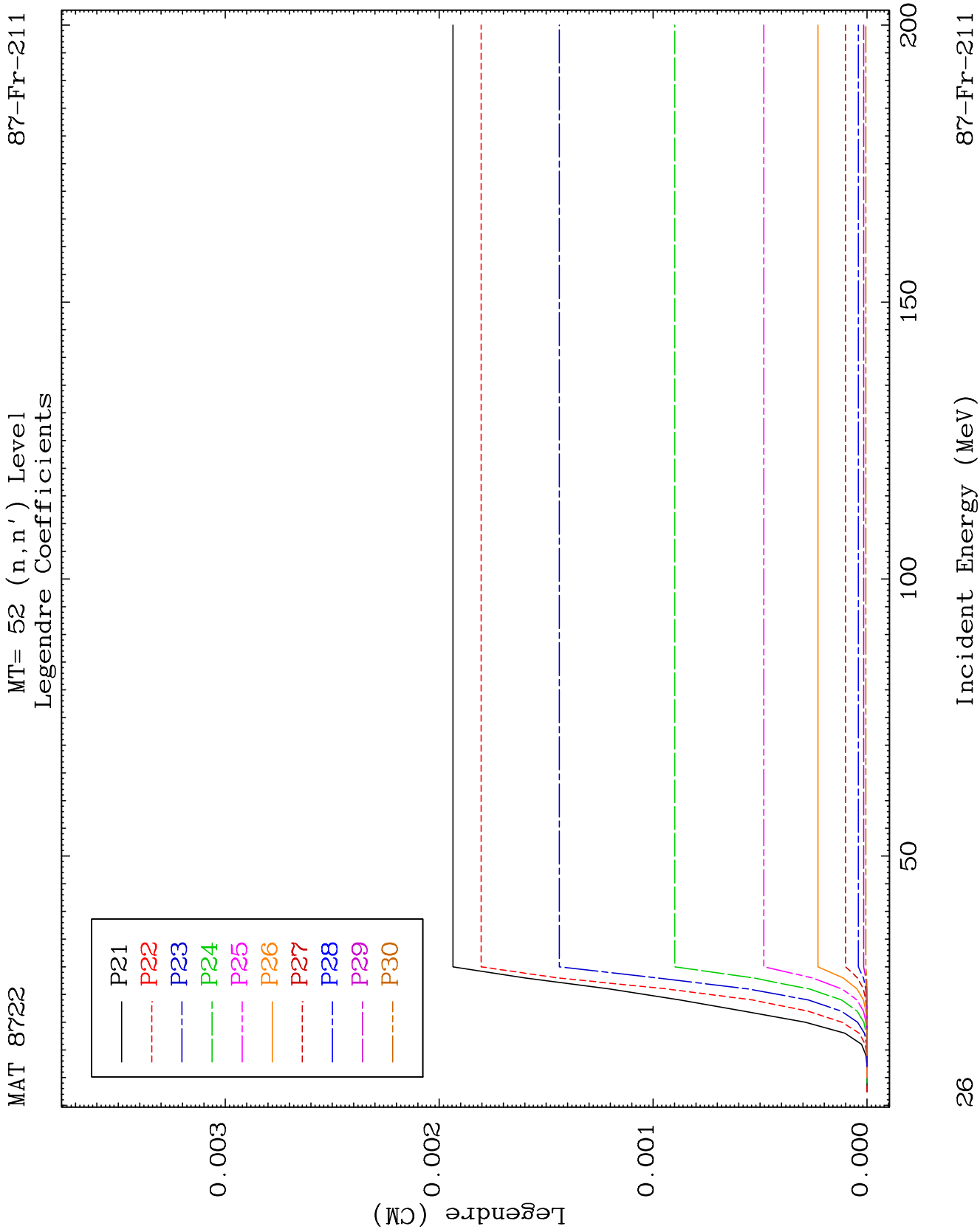


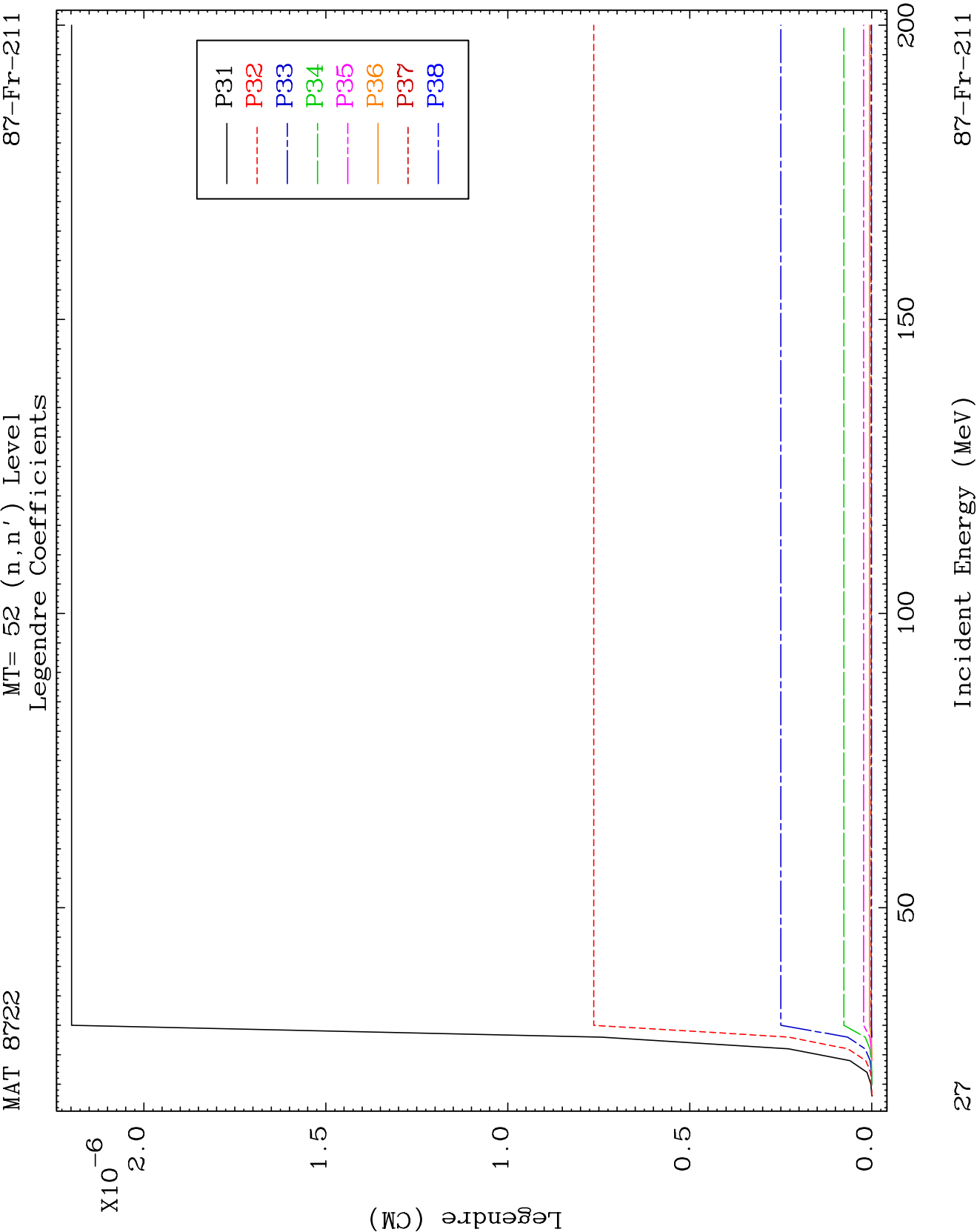
MAT 8722

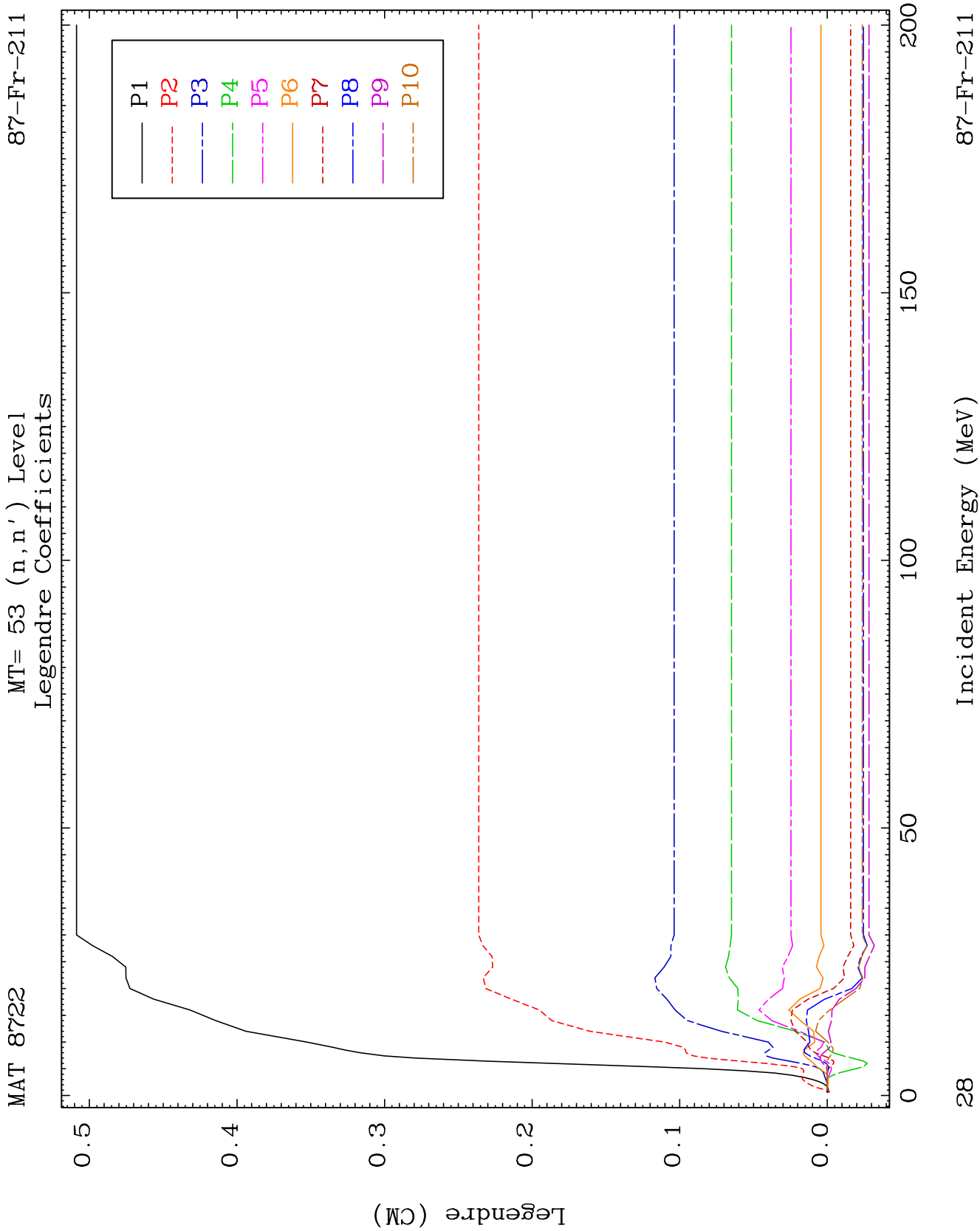
MT= 52 (n, n') Level
Legendre Coefficients

87-Fr-211





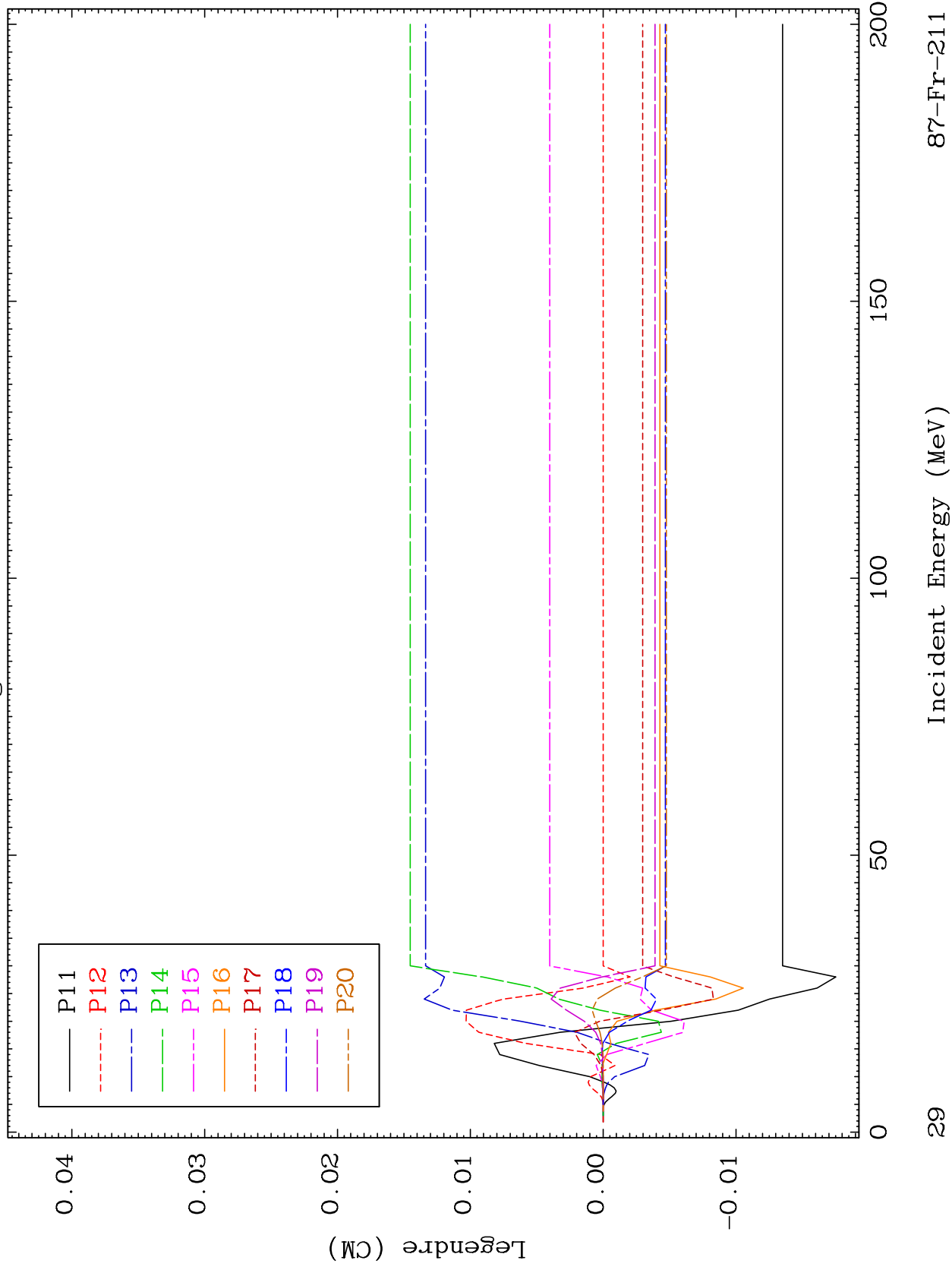


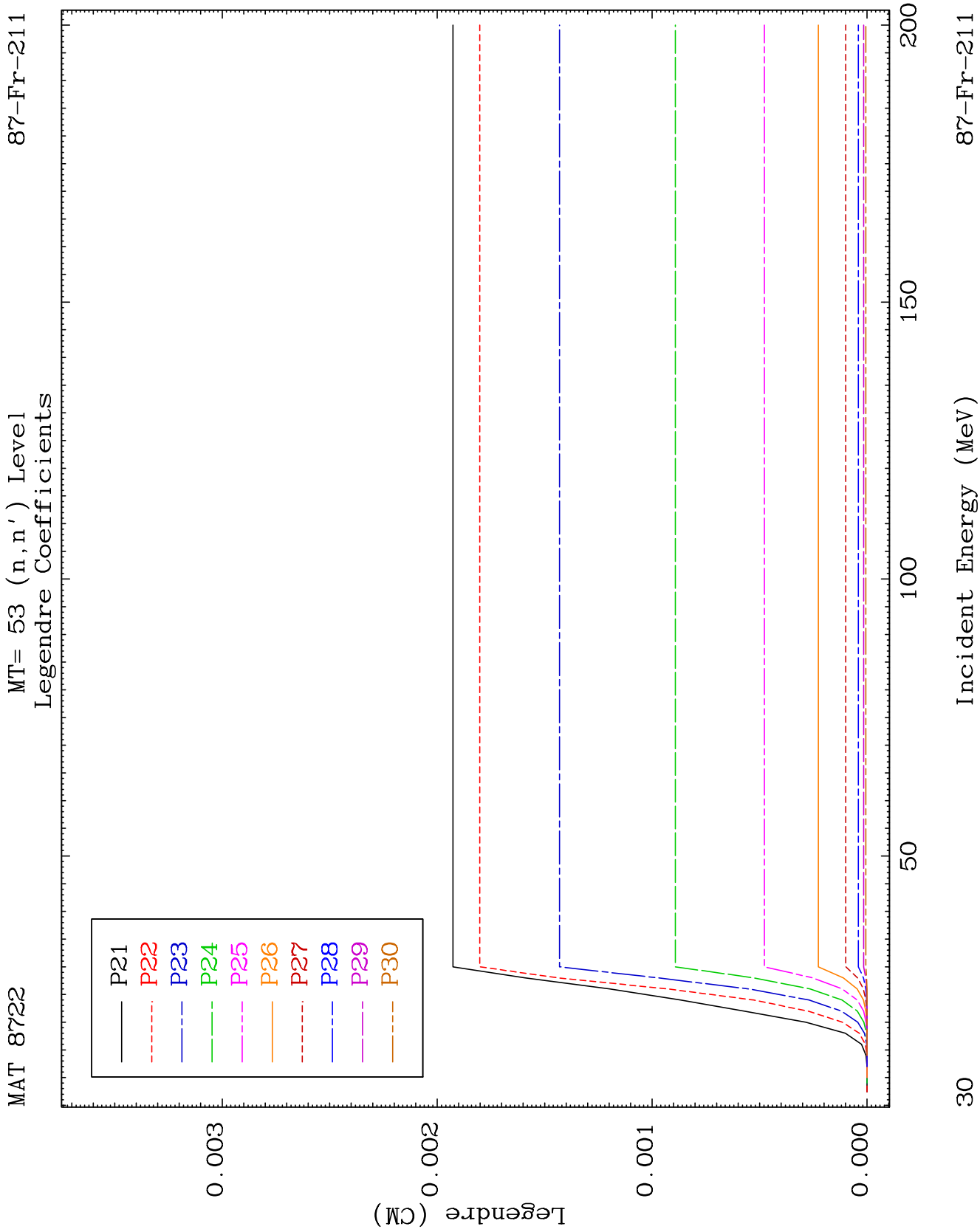


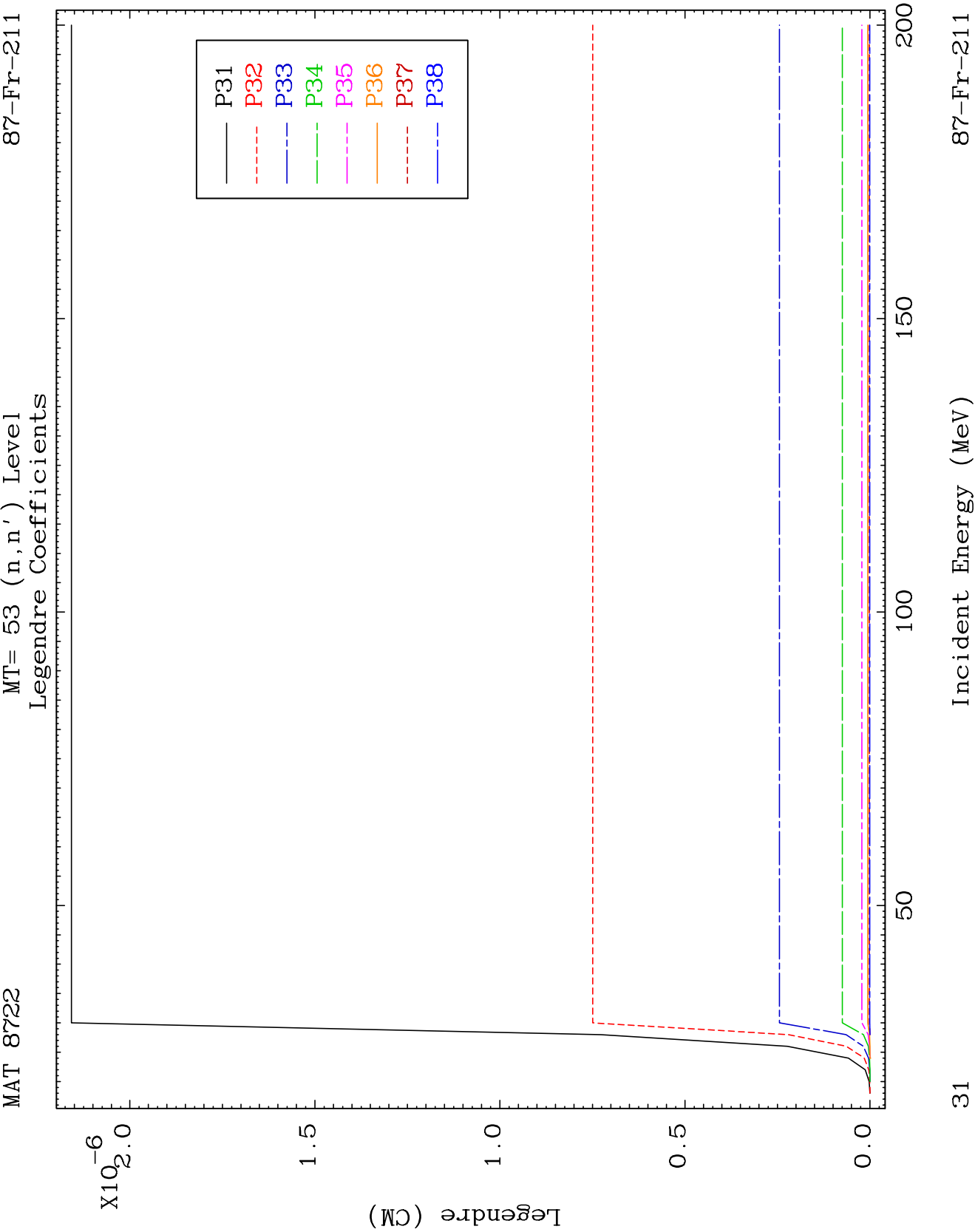
MAT 8722

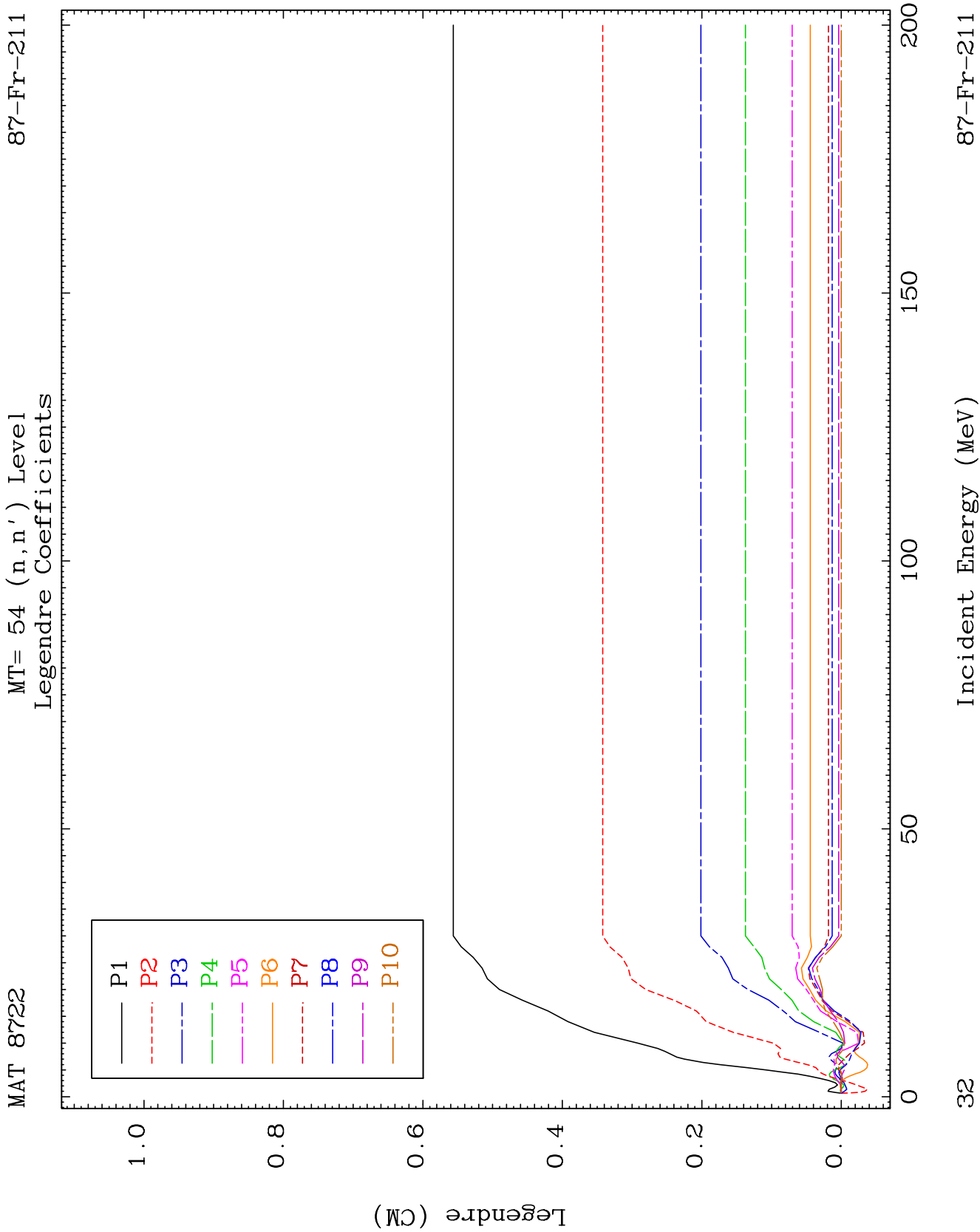
MT= 53 (n, n') Level
Legendre Coefficients

87-Fr-211





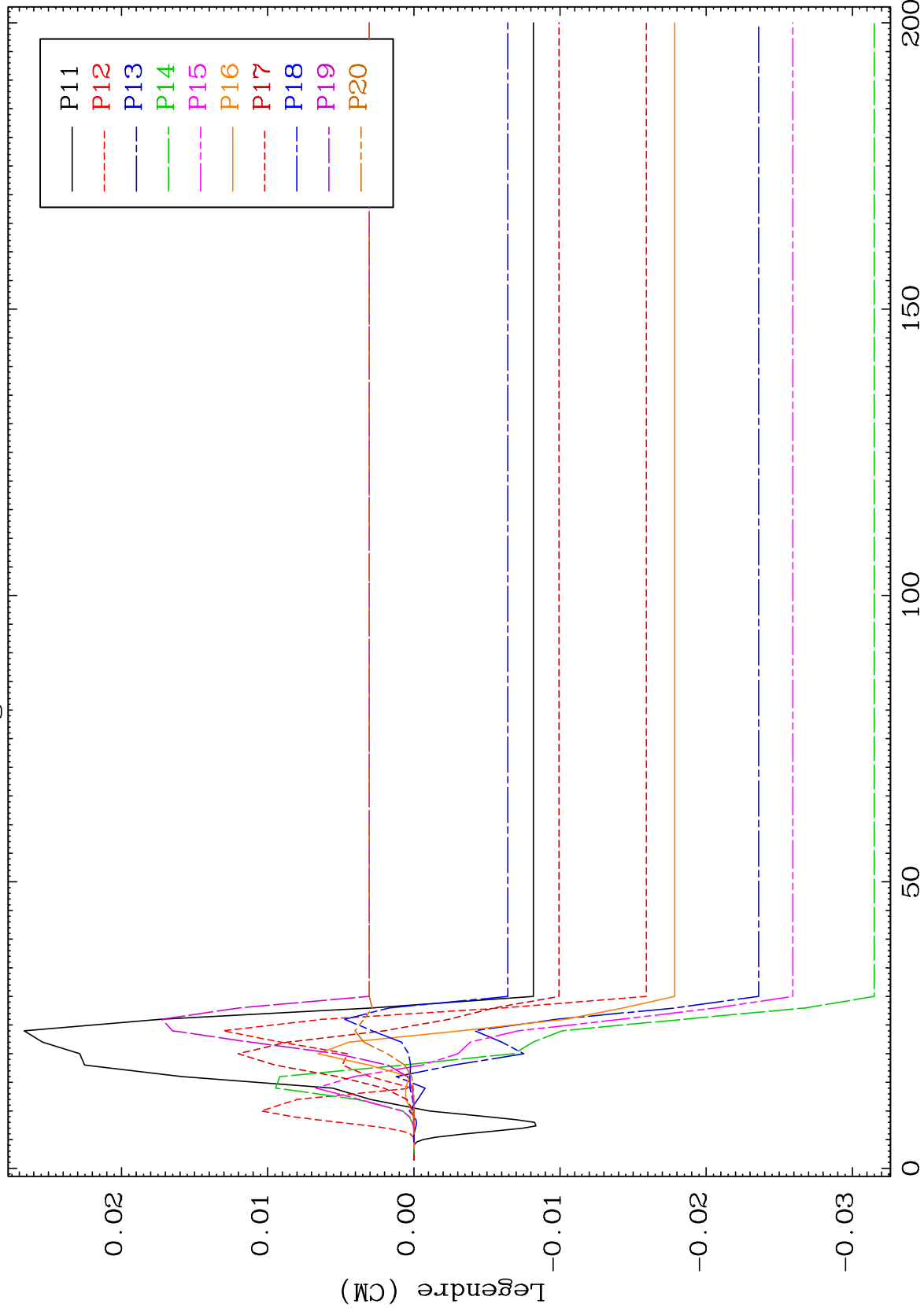




MAT 87222

MT= 54 (n,n') Level
Legendre Coefficients

87-Fr-211



87-Fr-211

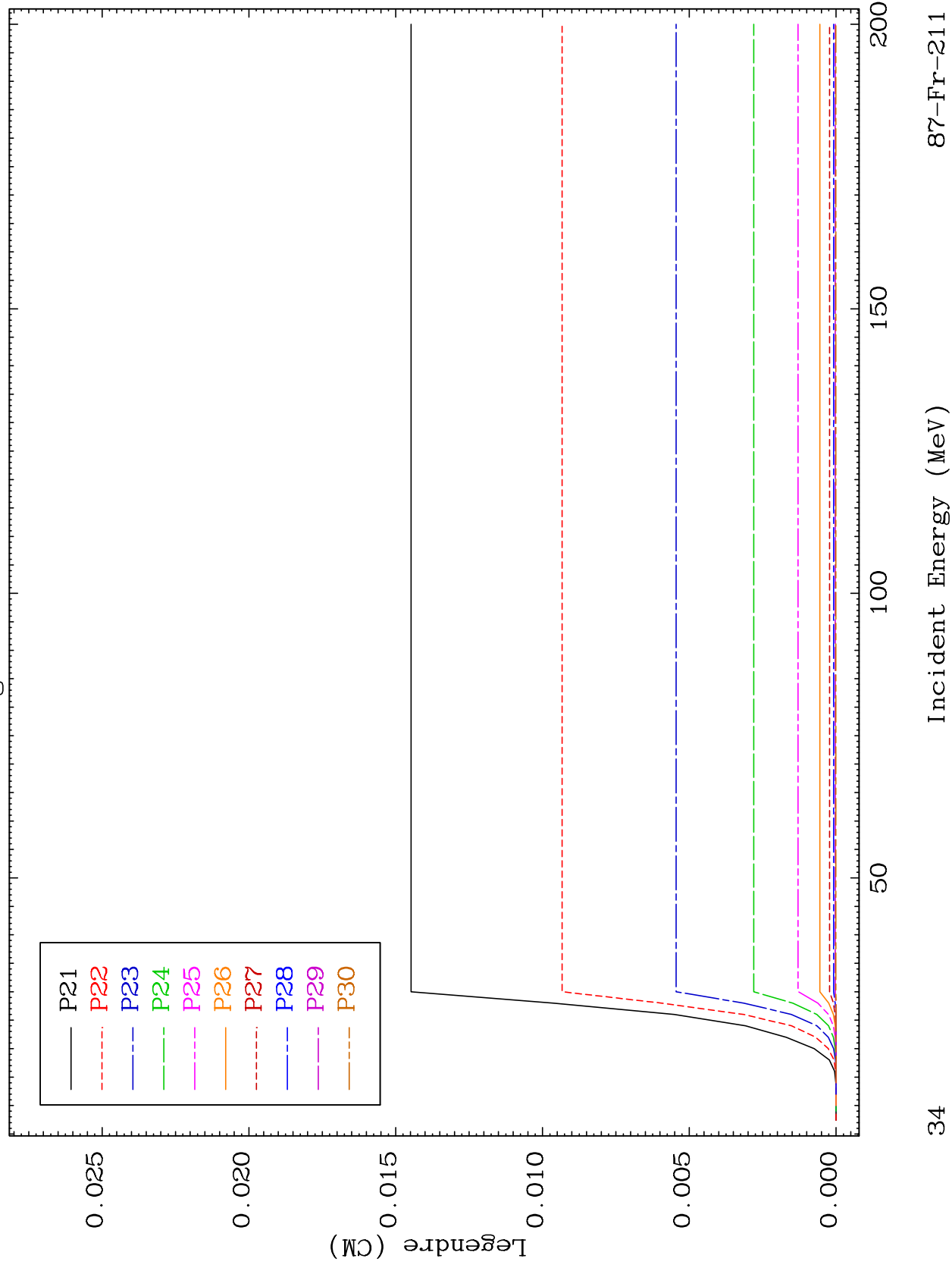
Incident Energy (MeV)

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MAT 8722

MT= 54 (n, n') Level
Legendre Coefficients

87-Fr-211

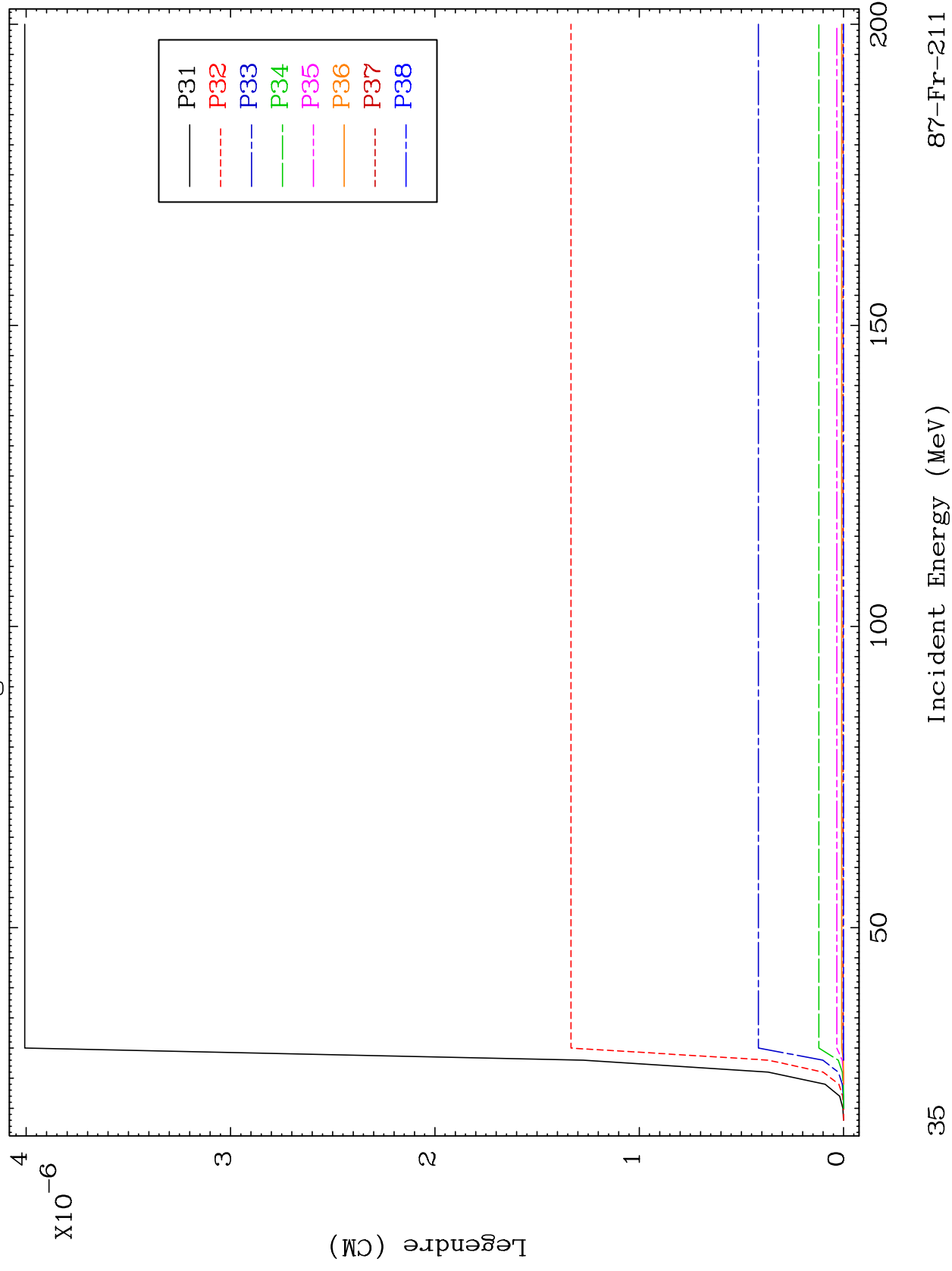


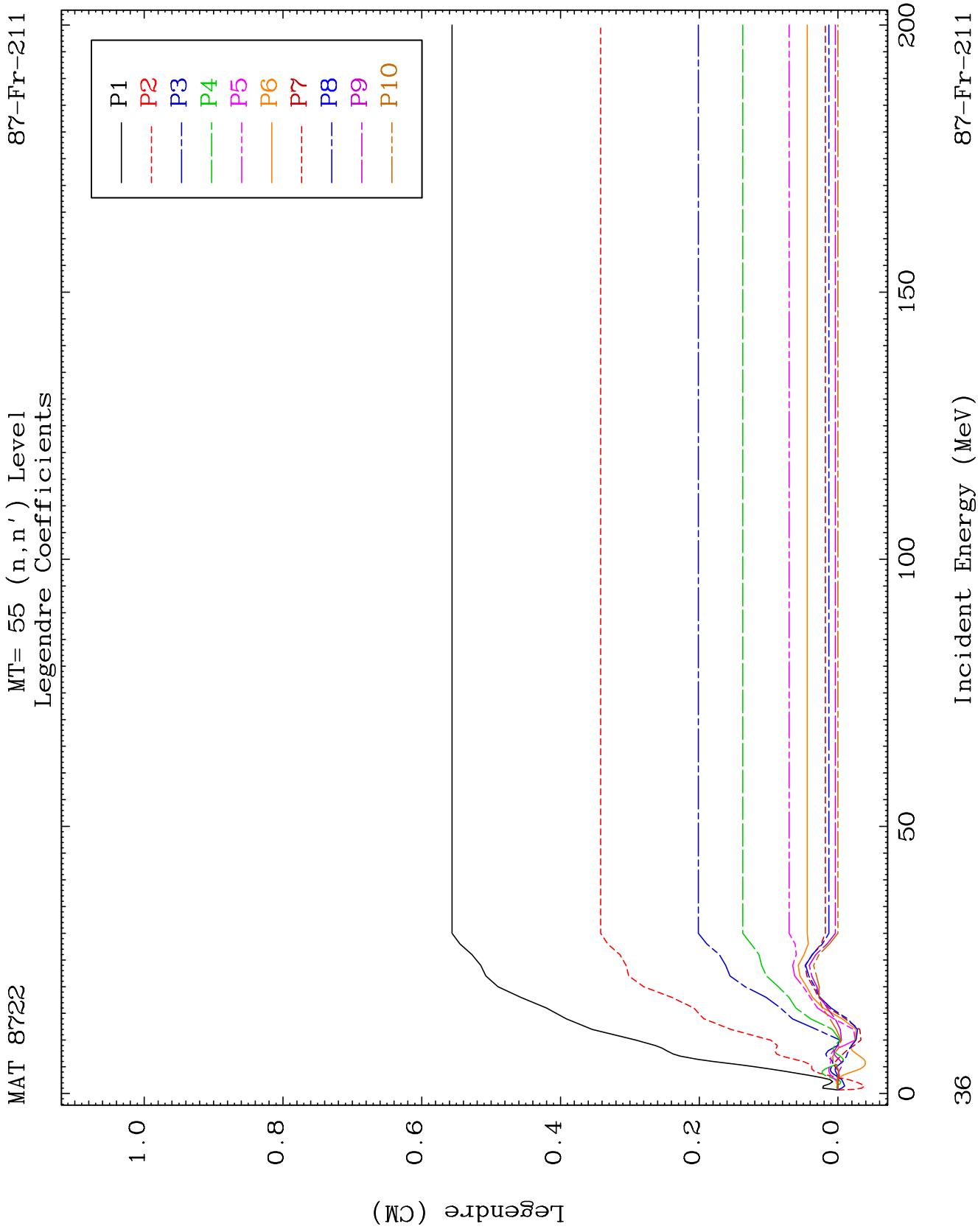
MAT 8722

MT=54 (n,n') Level

87-Fr-211

Legendre Coefficients

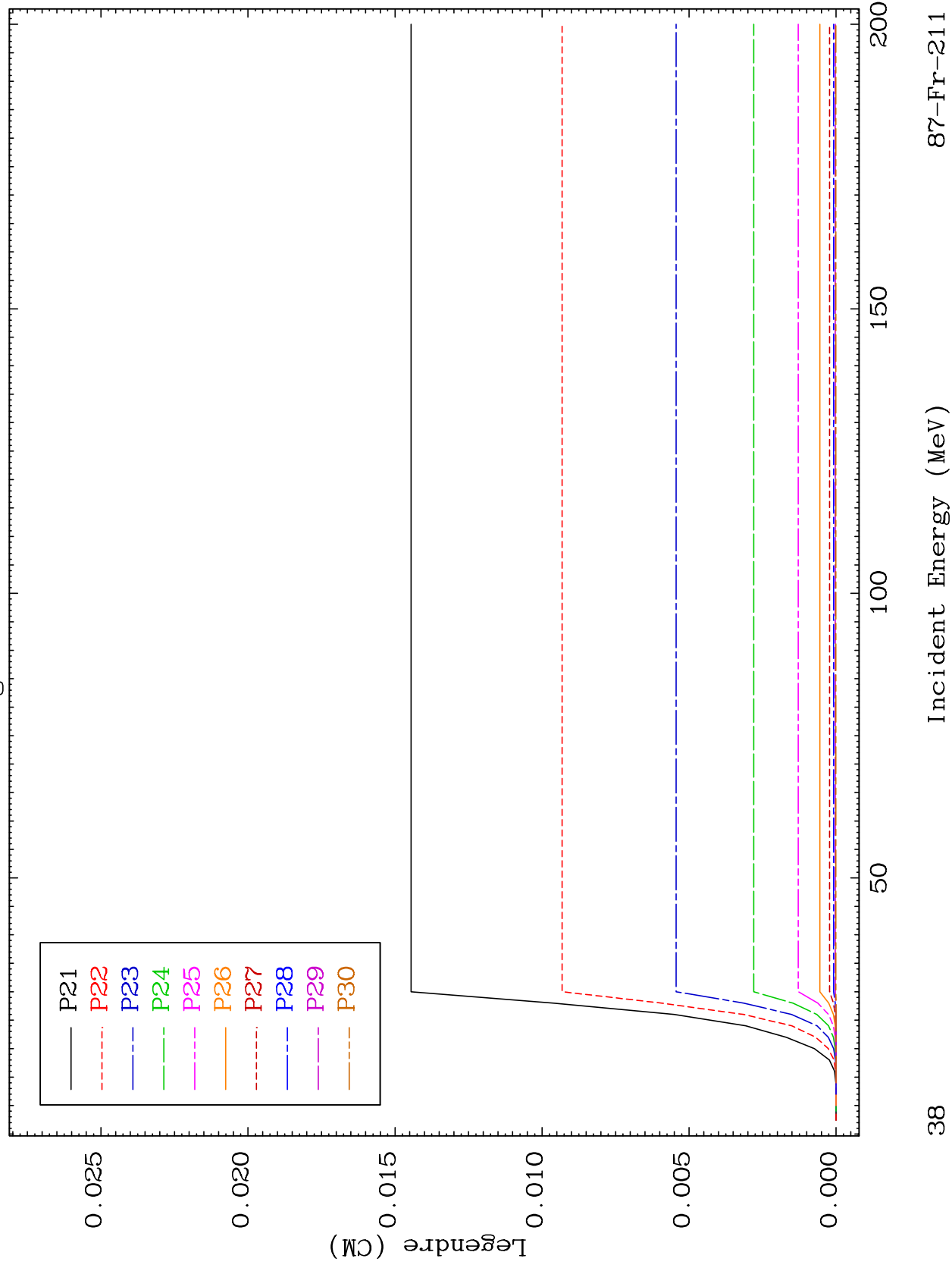


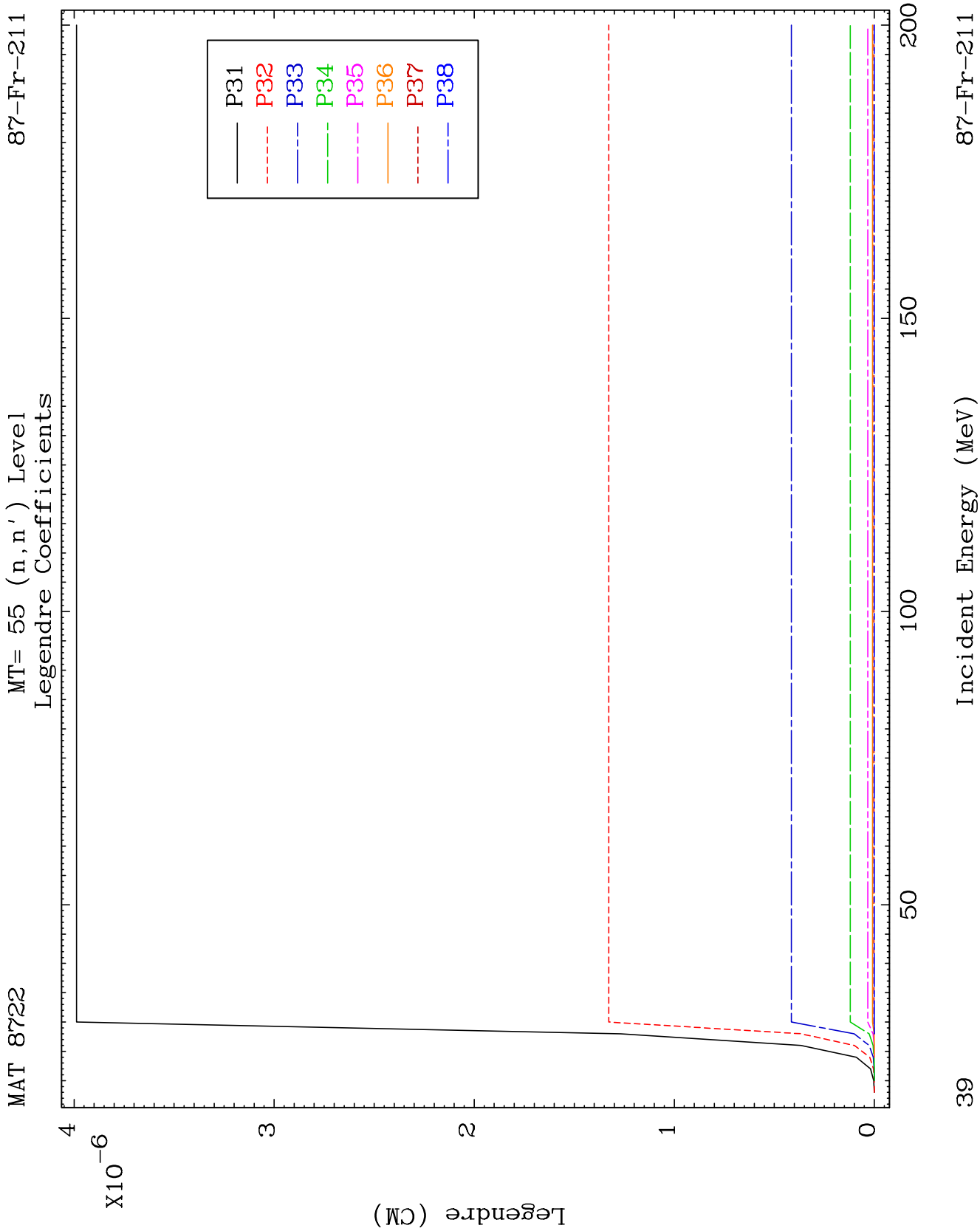


MAT 8722

MT= 55 (n, n') Level
Legendre Coefficients

87-Fr-211

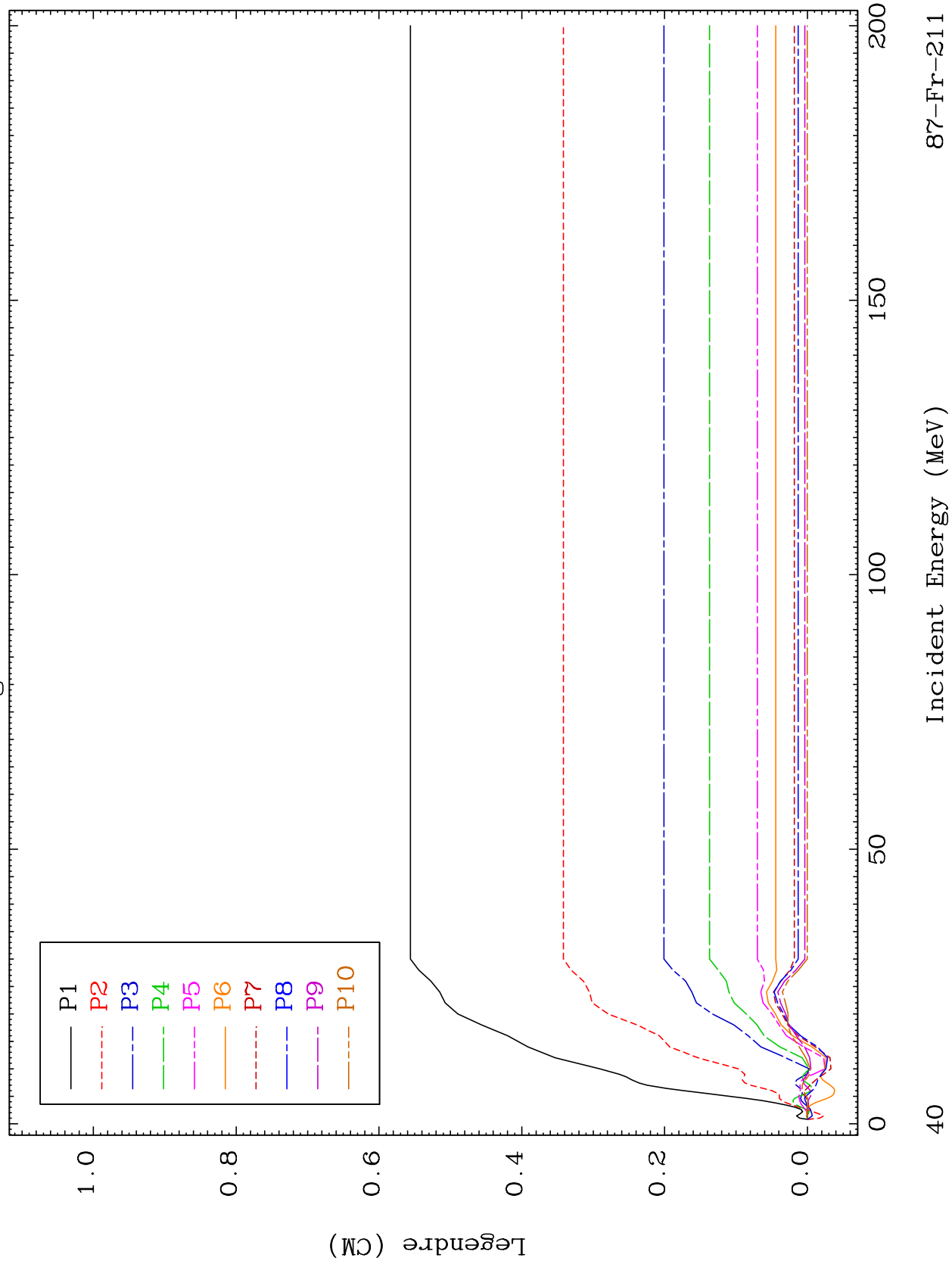


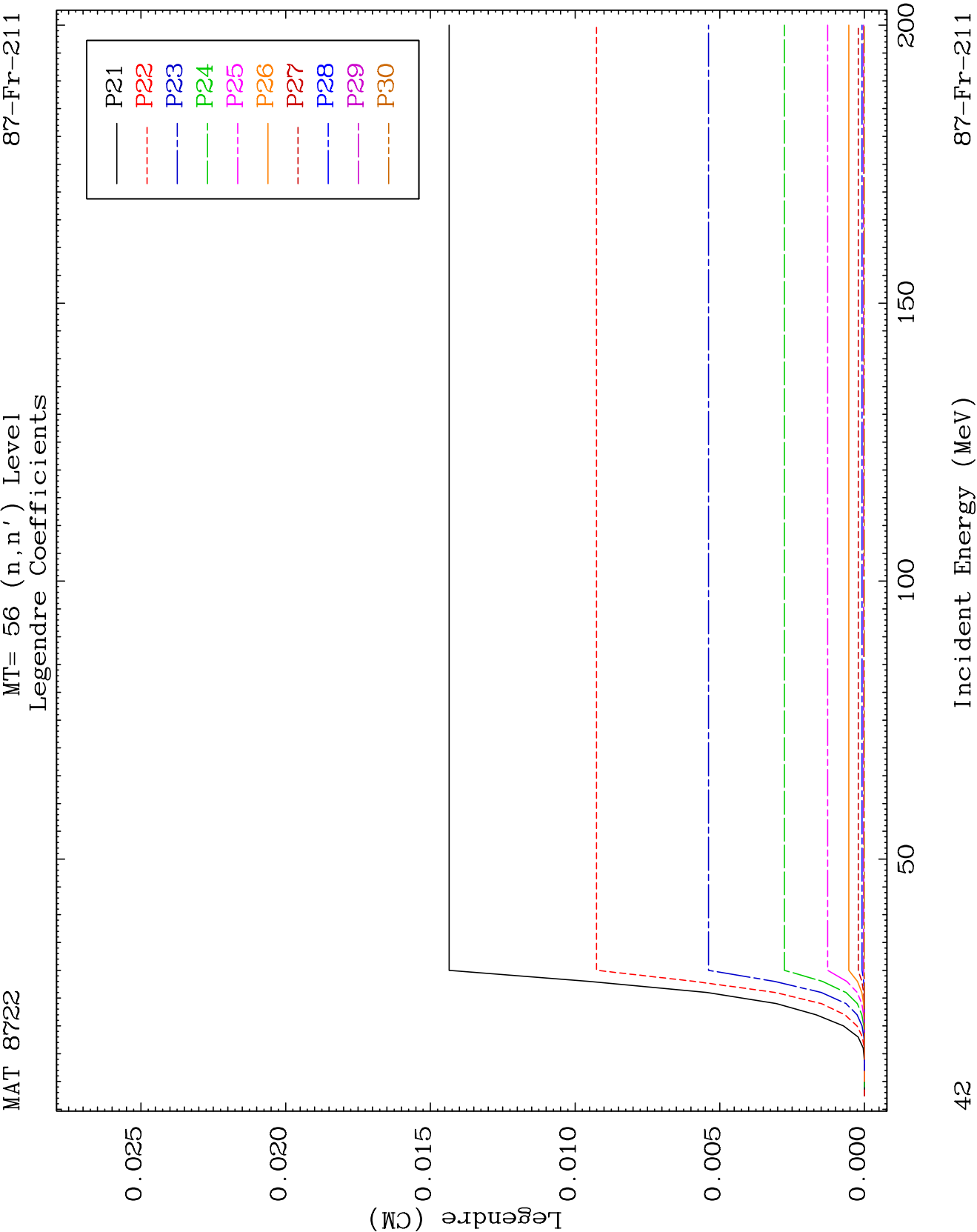


MAT 8722

MT= 56 (n, n') Level
Legendre Coefficients

87-Fr-211

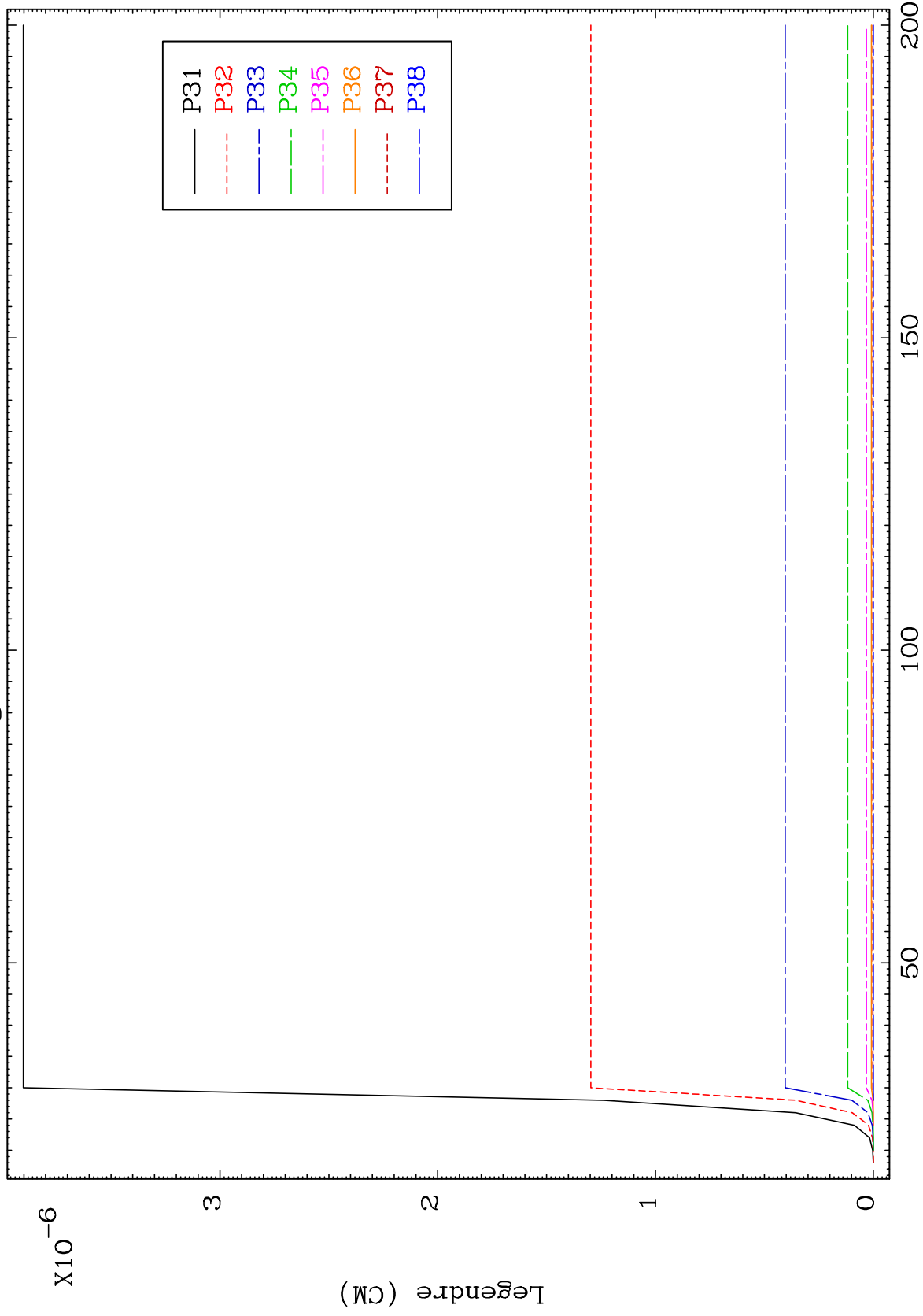




MAT 8722

MT= 56 (n,n') Level
Legendre Coefficients

87-Fr-211



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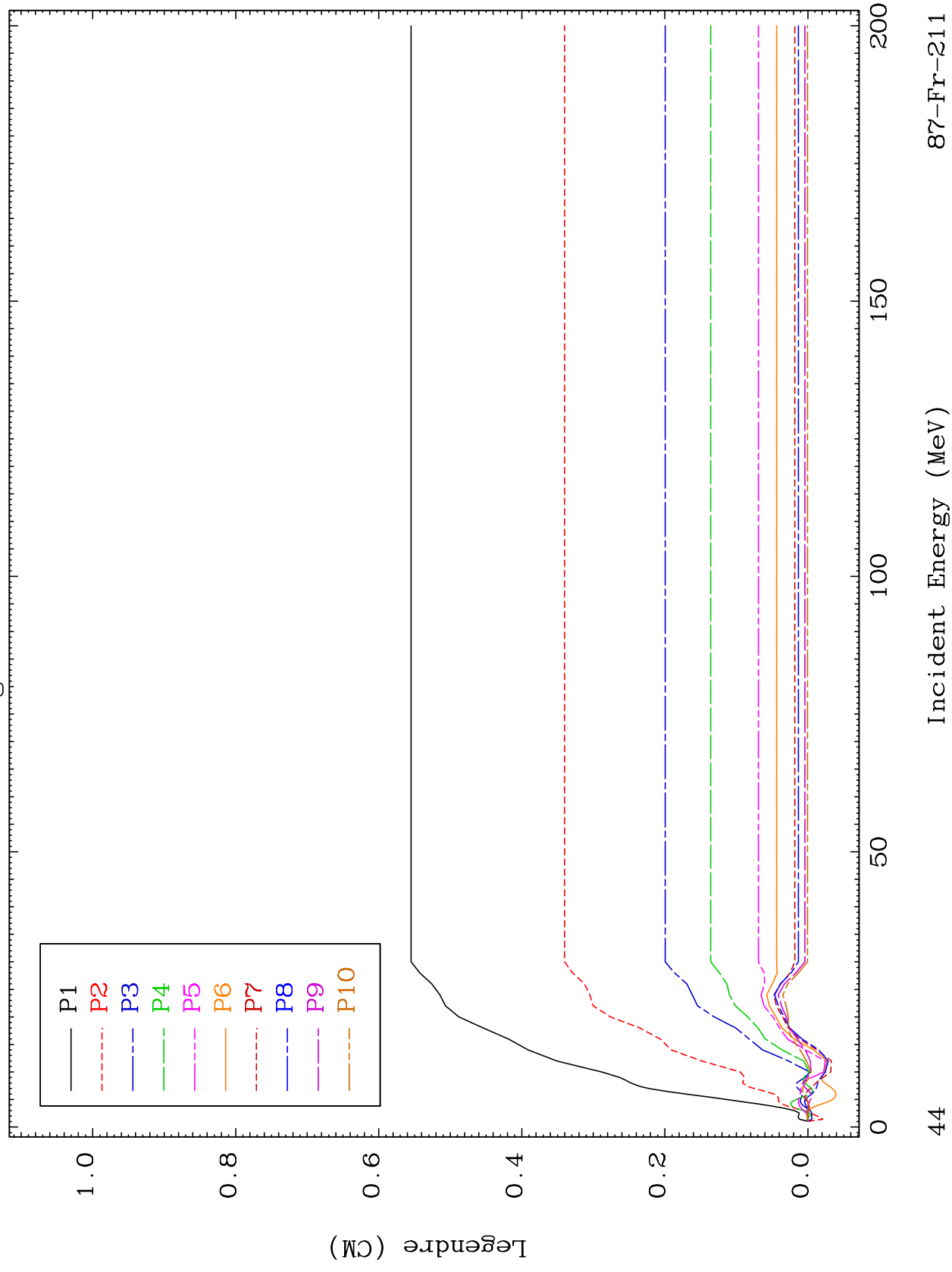
Incident Energy (MeV)

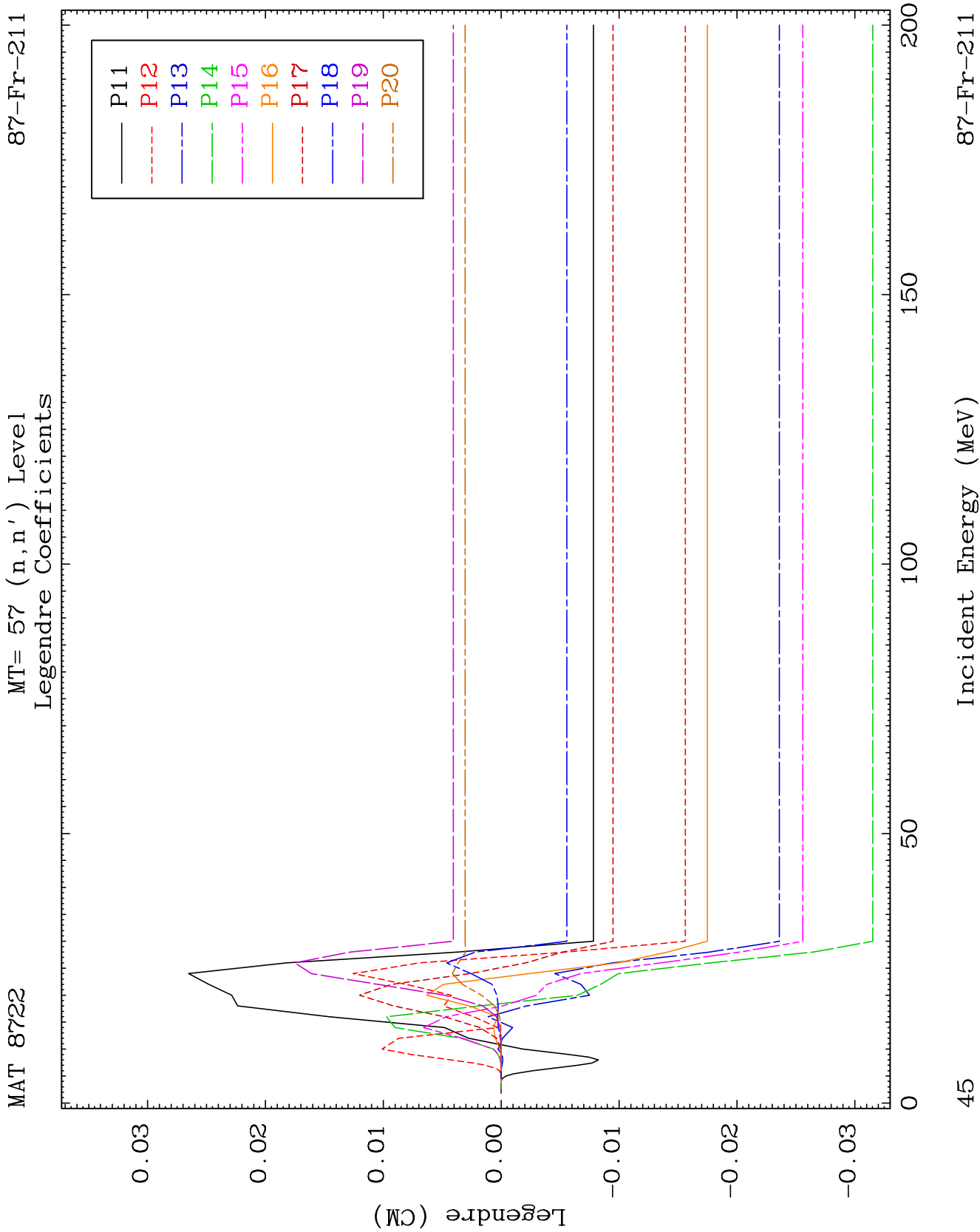
87-Fr-211

MAT 8722

MT= 57 (n, n') Level
Legendre Coefficients

87-Fr-211

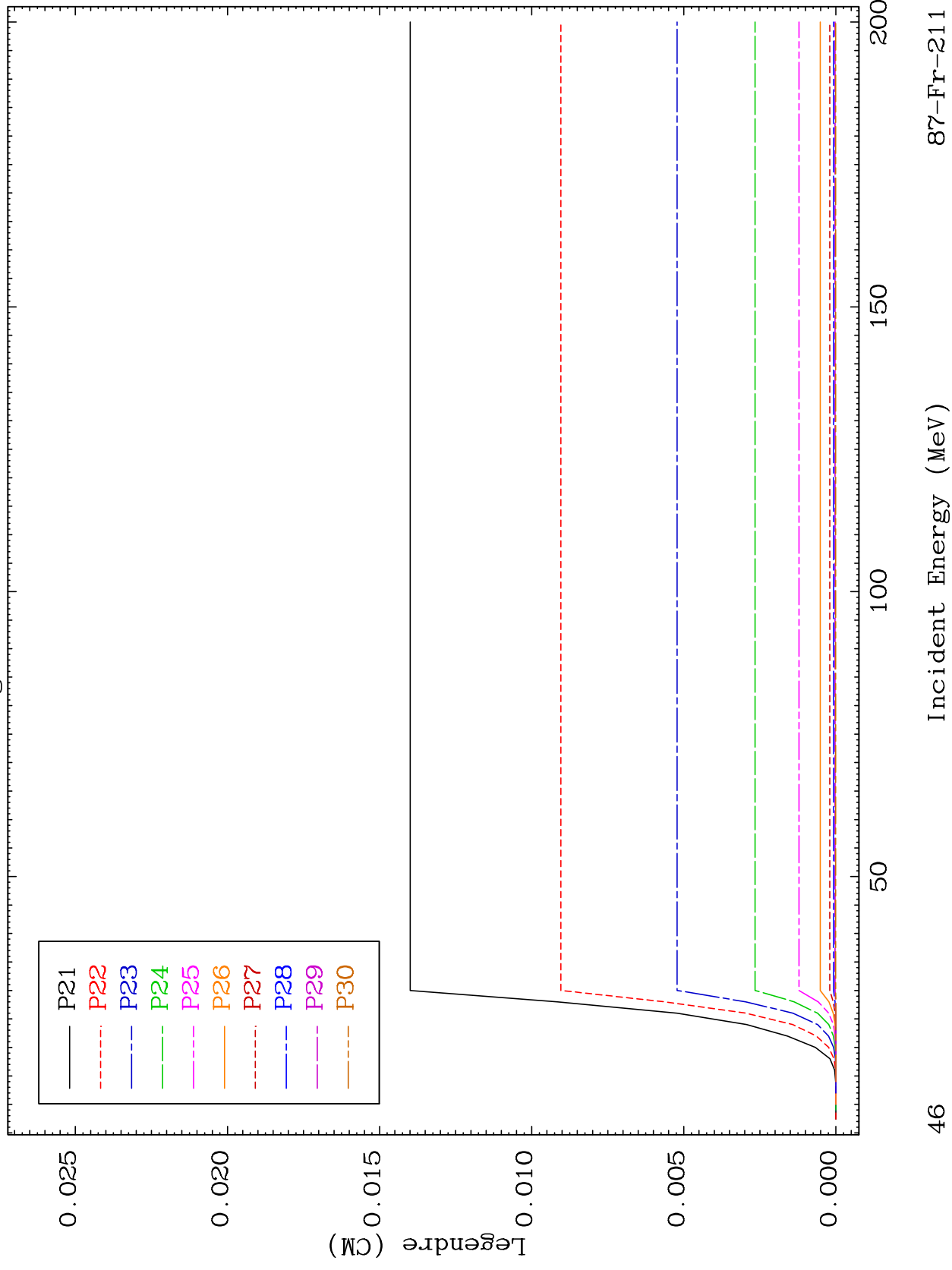




MAT 8722

MT= 57 (n, n') Level
Legendre Coefficients

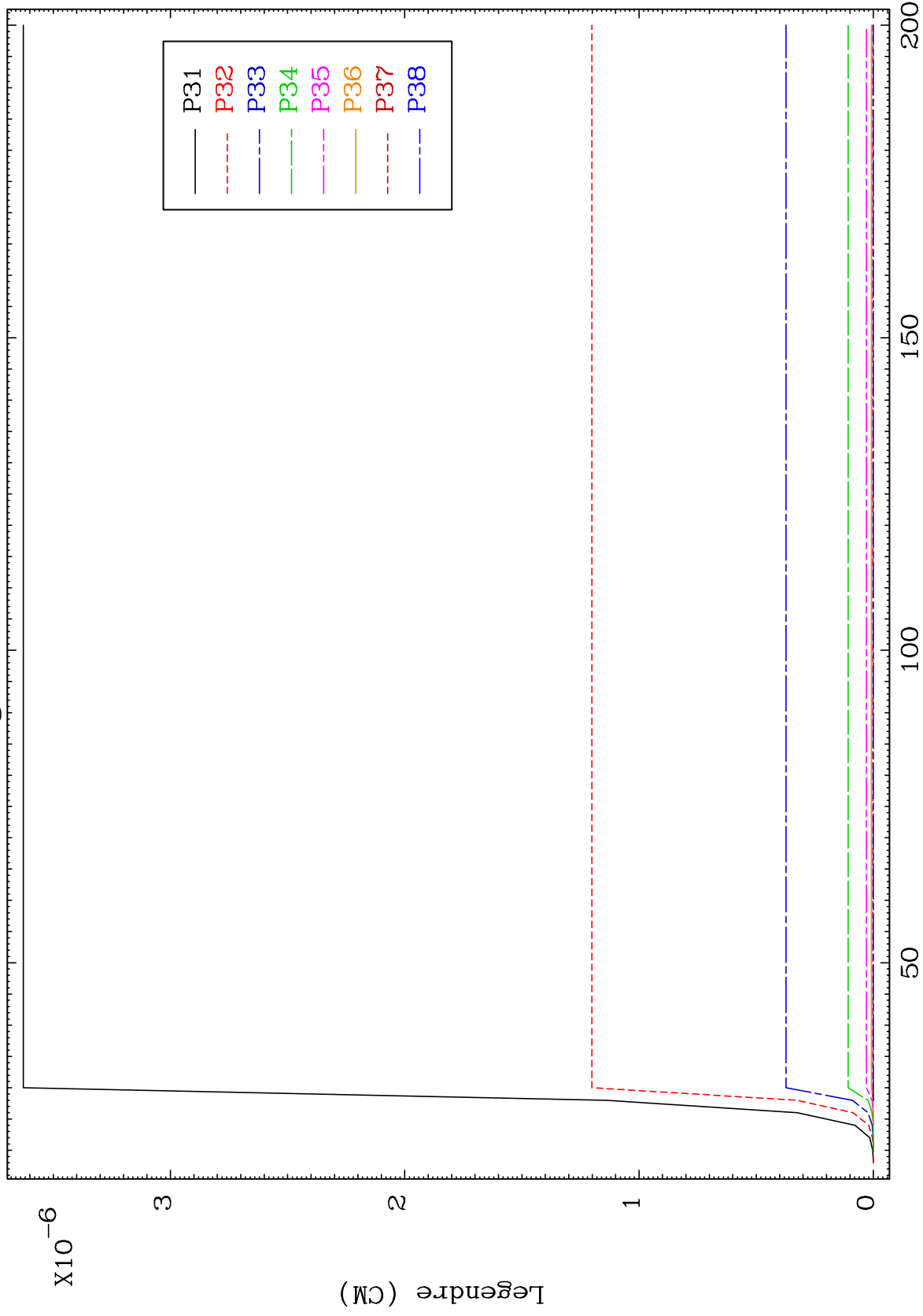
87-Fr-211



MAT 87222

MT= 57 (n,n') Level
Legendre Coefficients

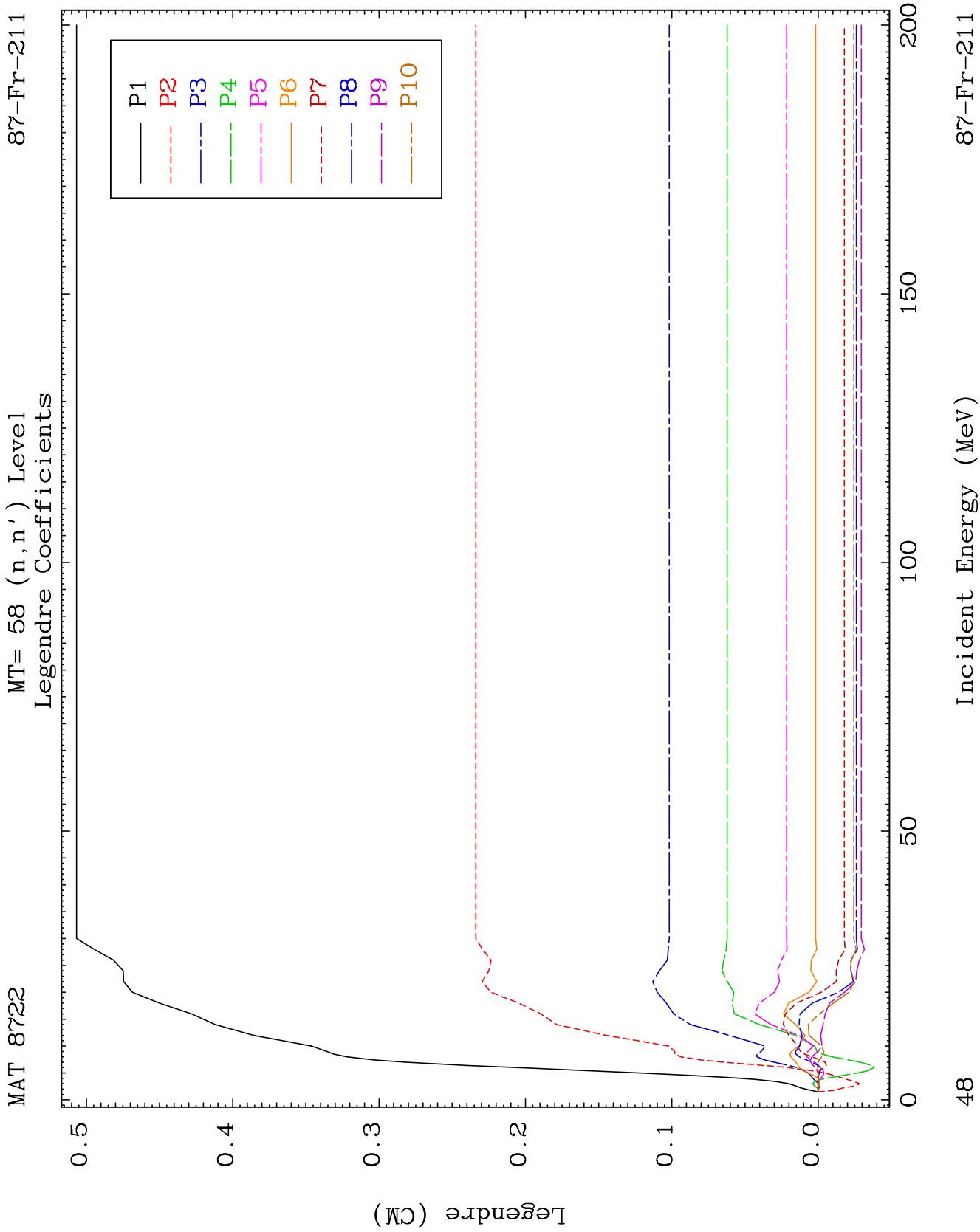
87-Fr-211



47

Incident Energy (MeV)

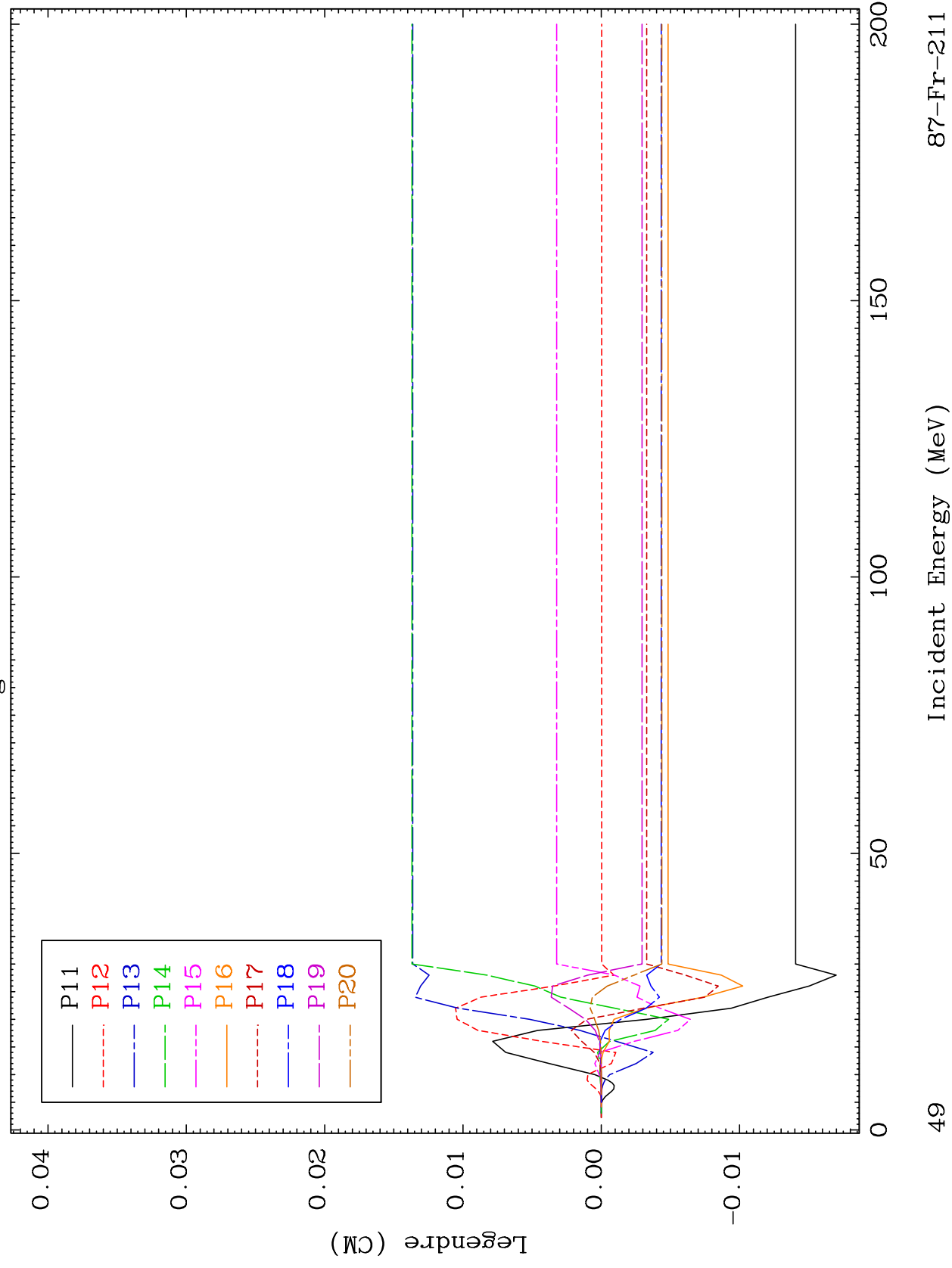
87-Fr-211



MAT 8722

MT= 58 (n, n') Level
Legendre Coefficients

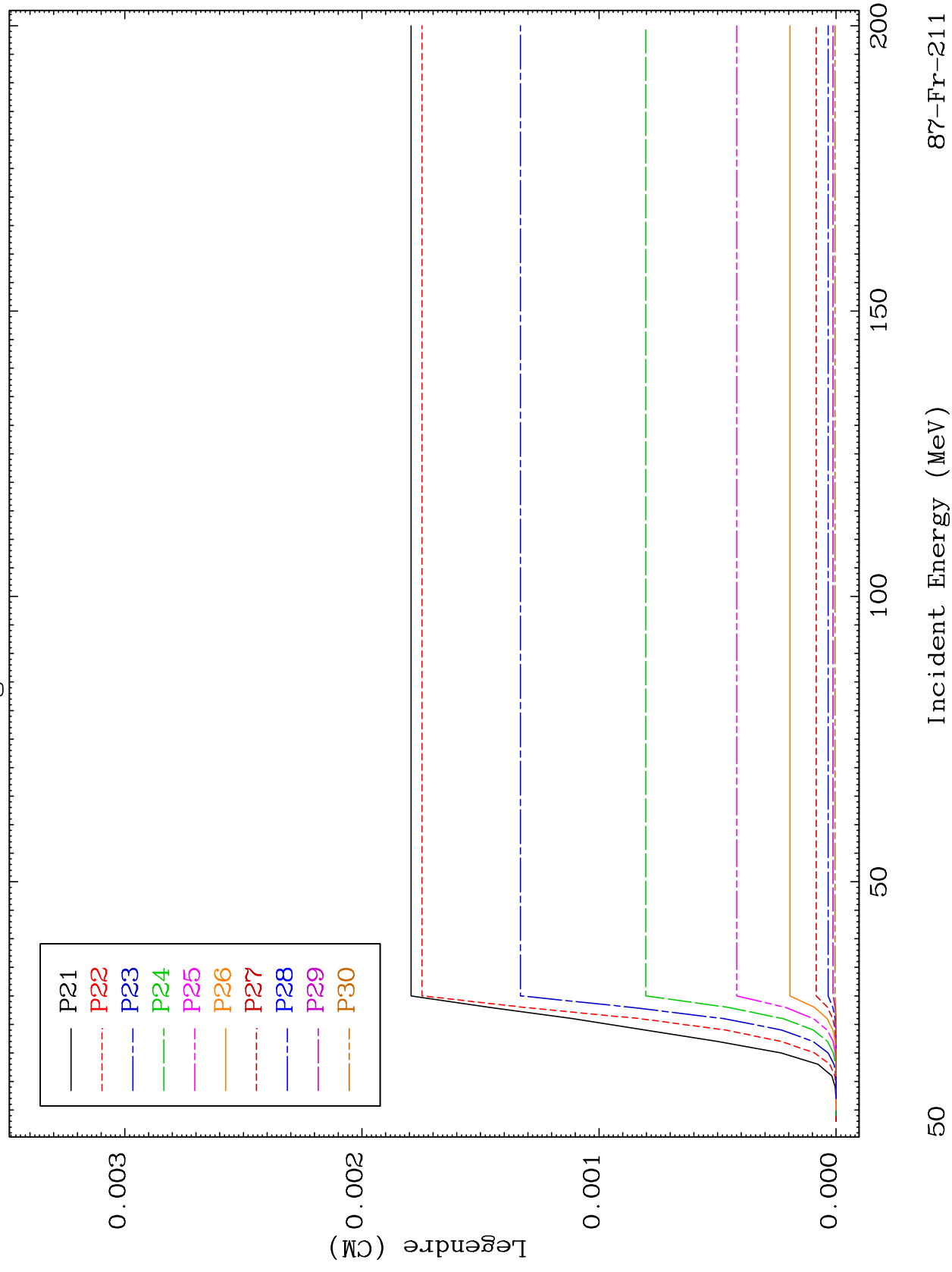
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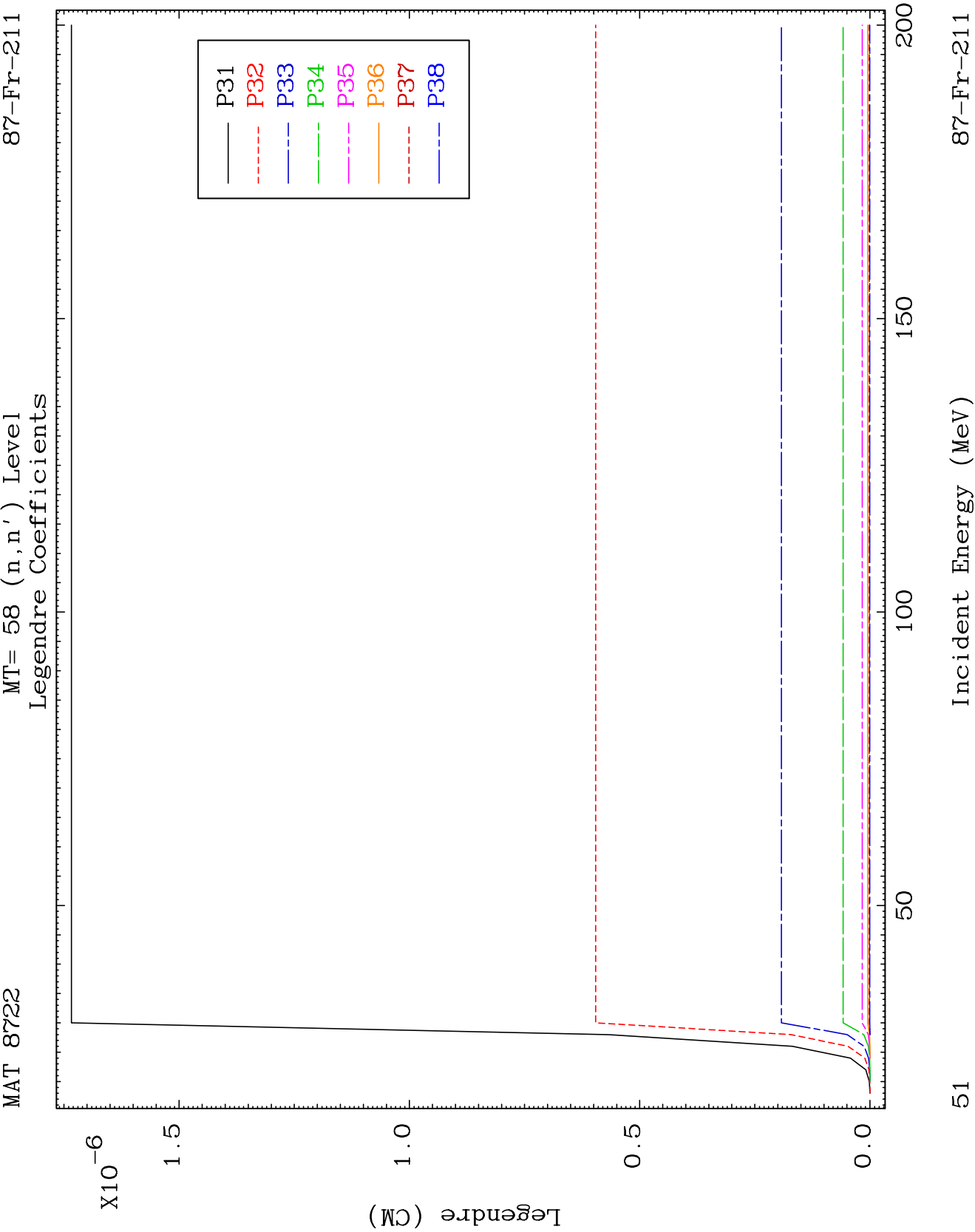


MAT 8722

MT= 58 (n, n') Level
Legendre Coefficients

87-Fr-211



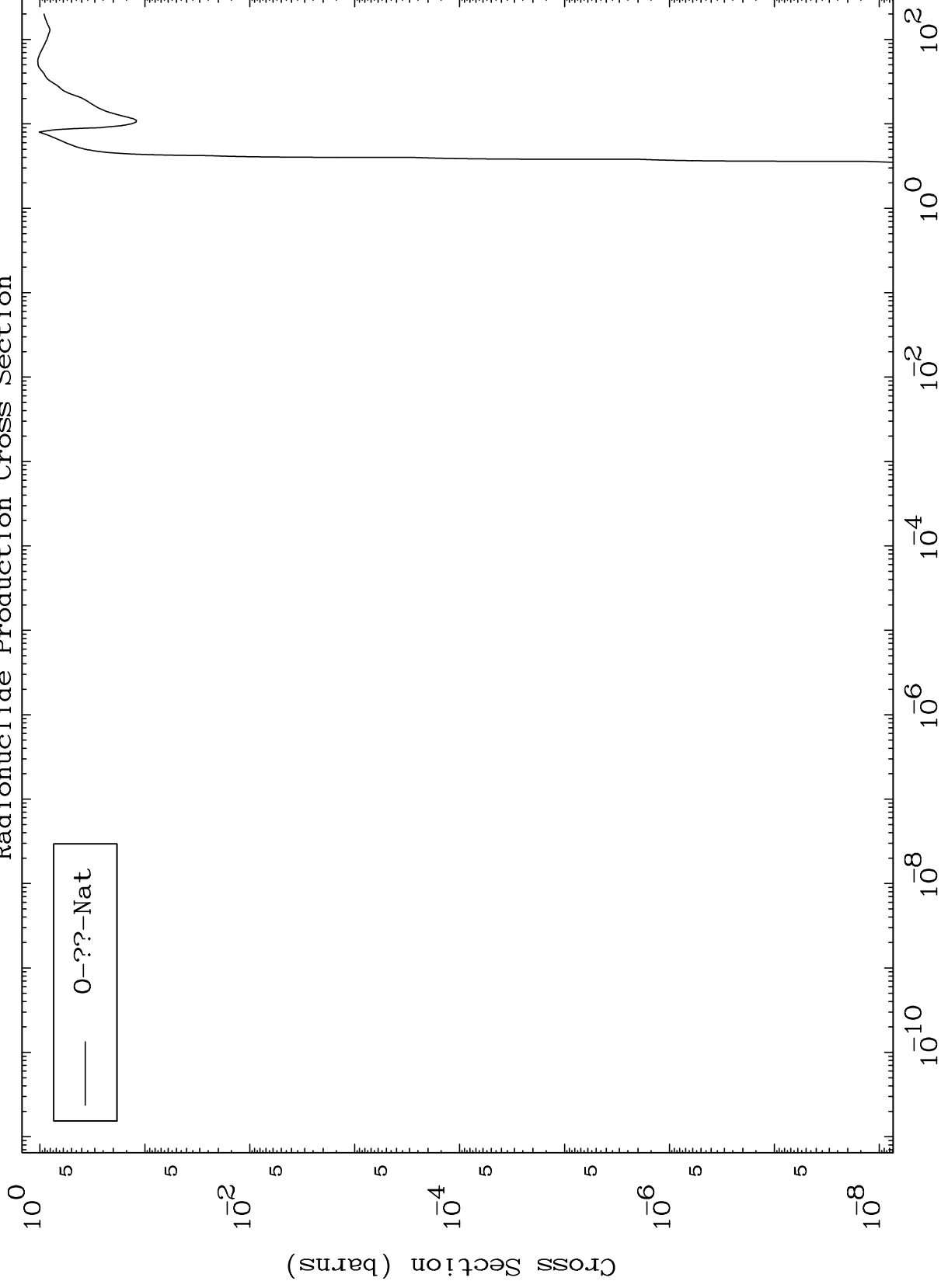


MAT 8722

Fission

87-Fr-211

Radionuclide Production Cross Section



52

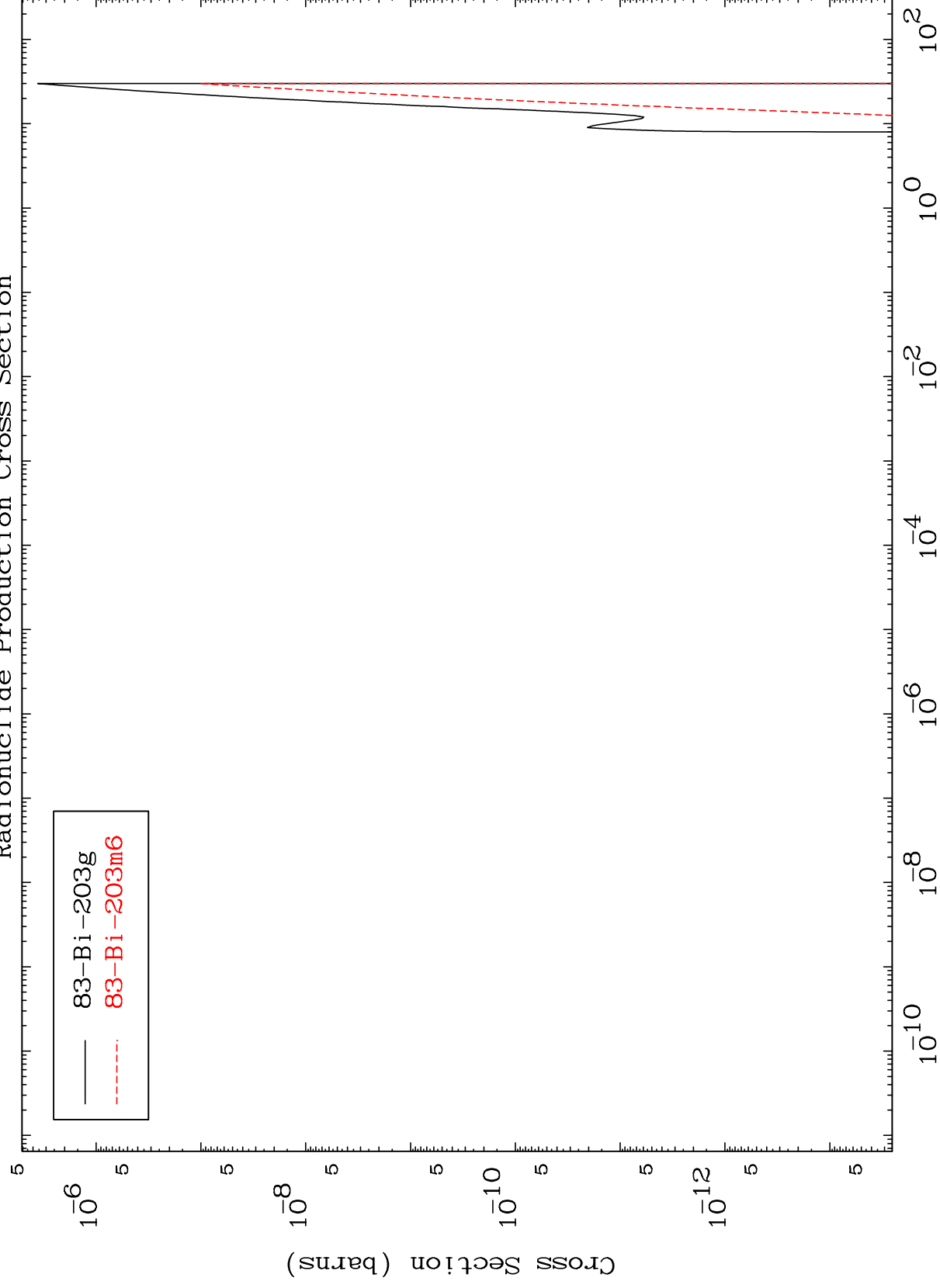
Incident Energy (MeV)

87-Fr-211

MAT 8722

87-Fr-211

(n,n') 2 α
Radionuclide Production Cross Section



53

87-Fr-211

(n,p) α
Radionuclide Production Cross Section

