

Program EVALPLOT  
(Version 2021-1)

by

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(Present Contact Information)

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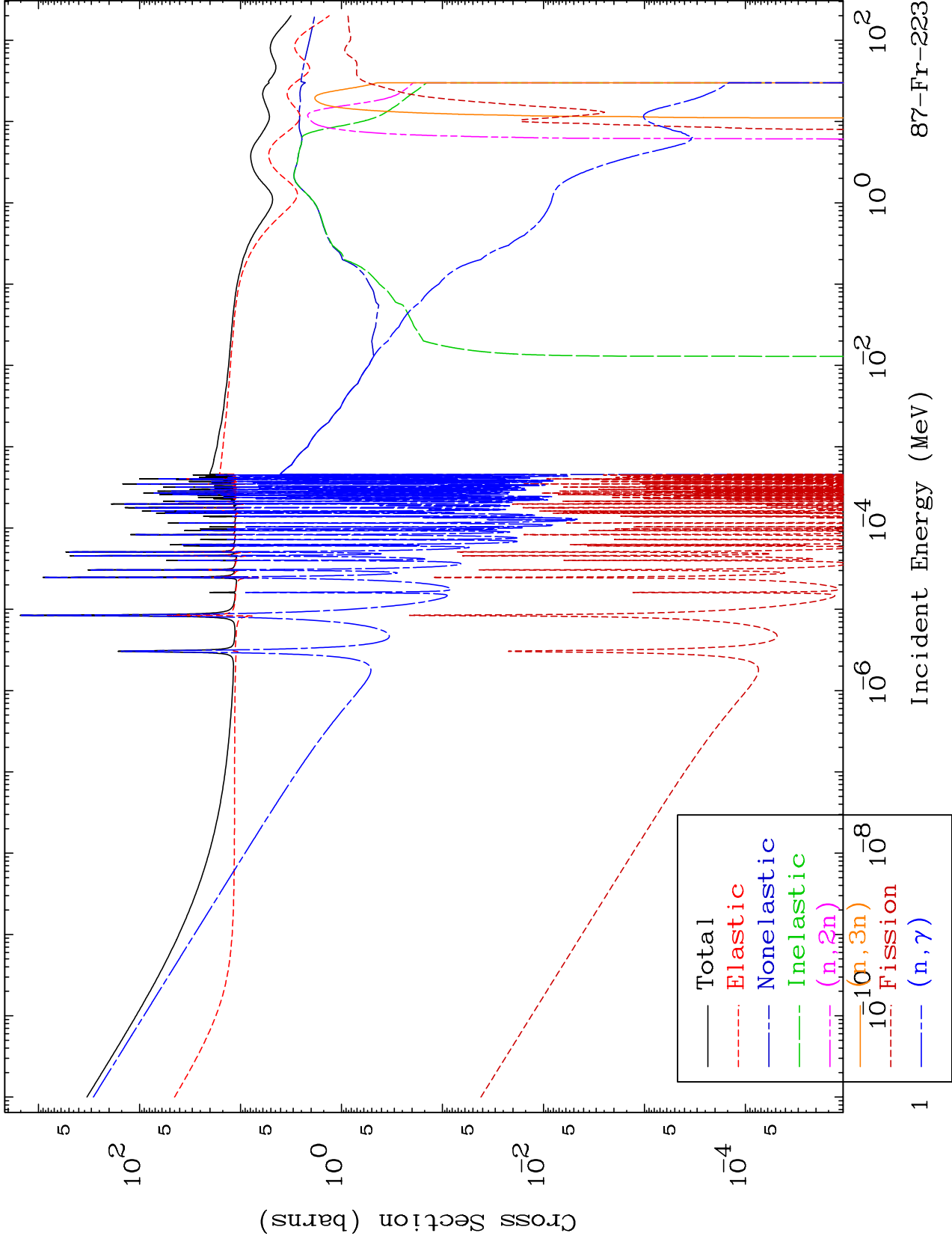
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

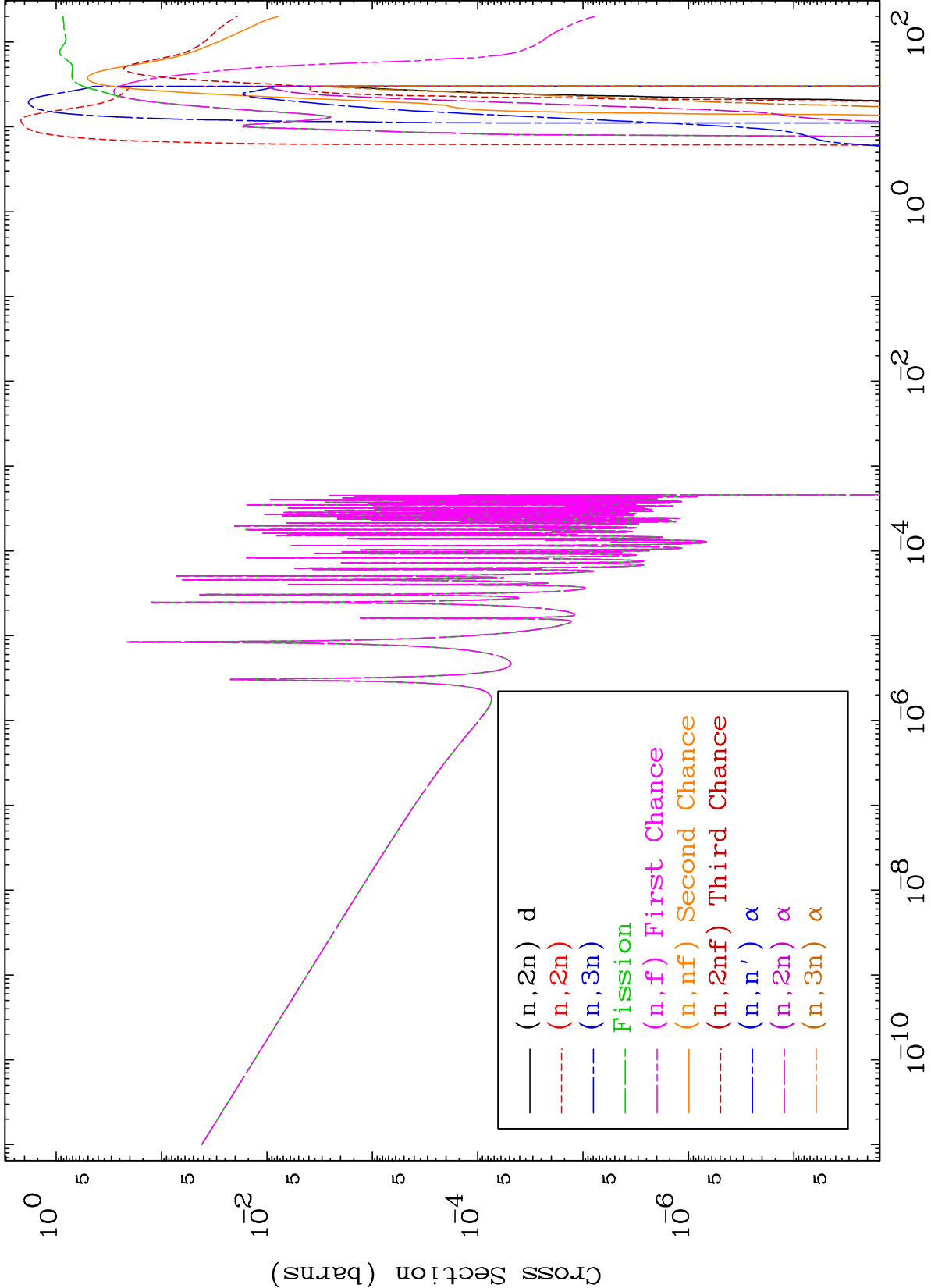
Press Mouse Button to Start

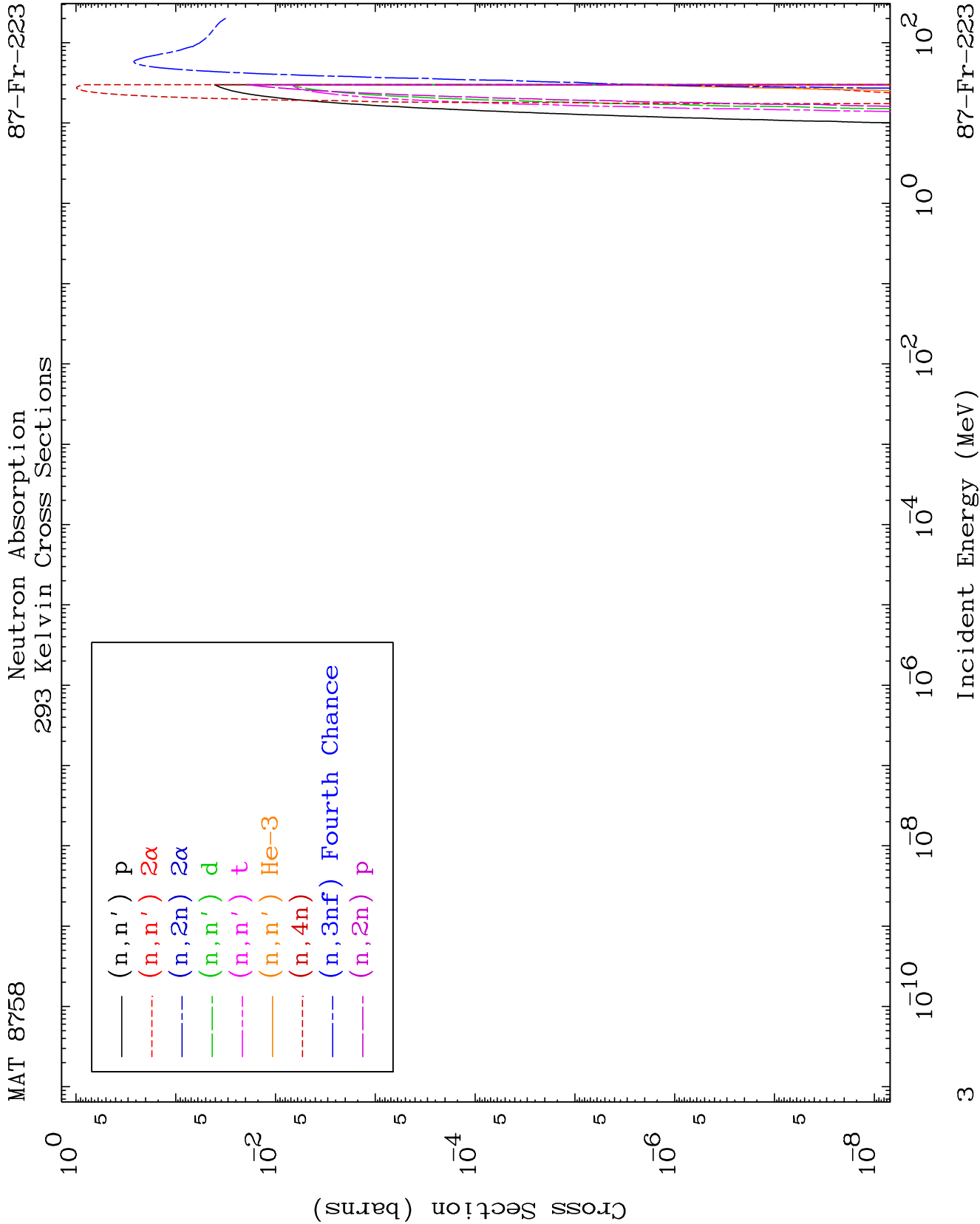
MAT 8758

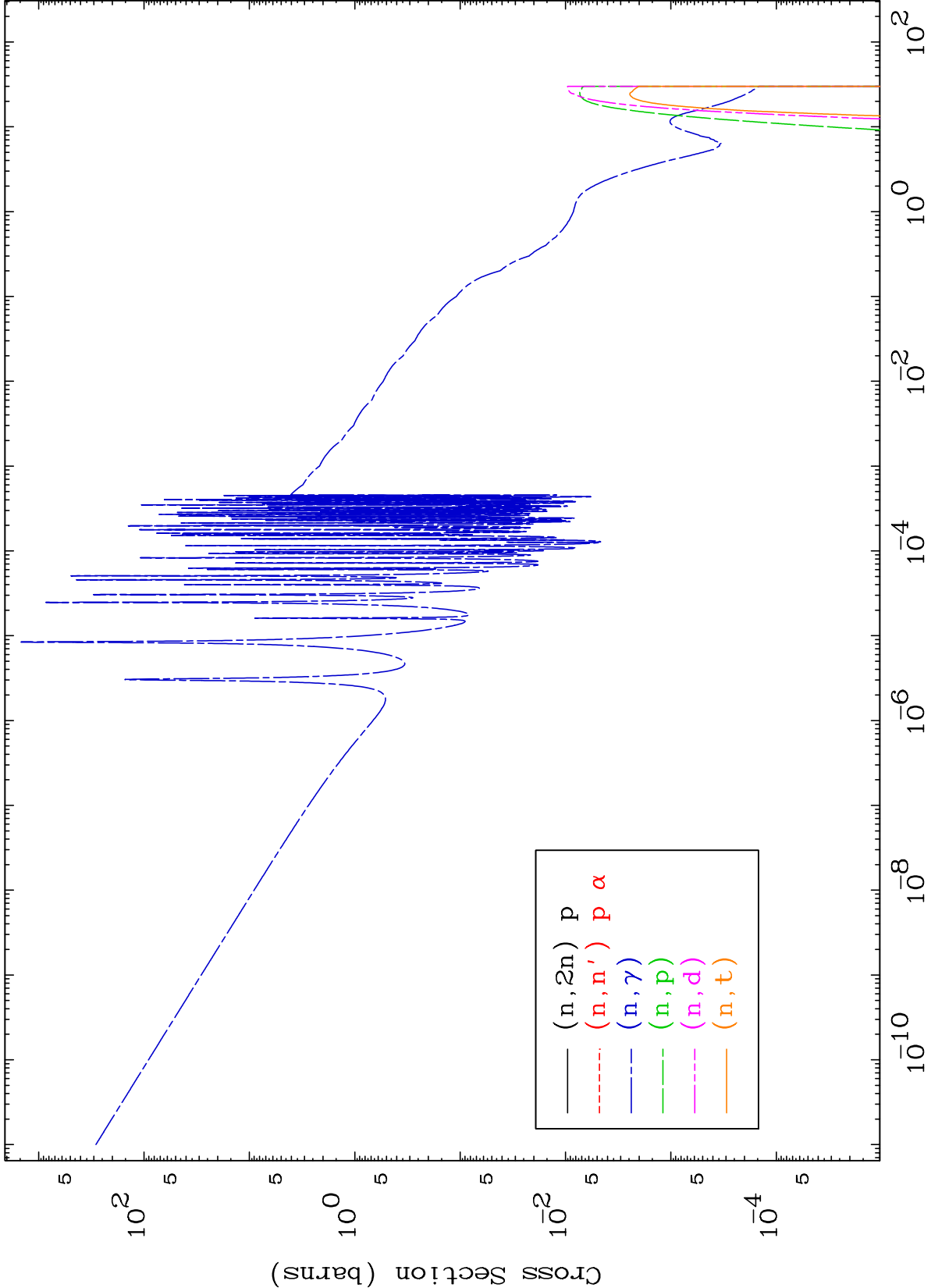
Neutron Major  
293 Kelvin Cross Sections

87-Fr-223





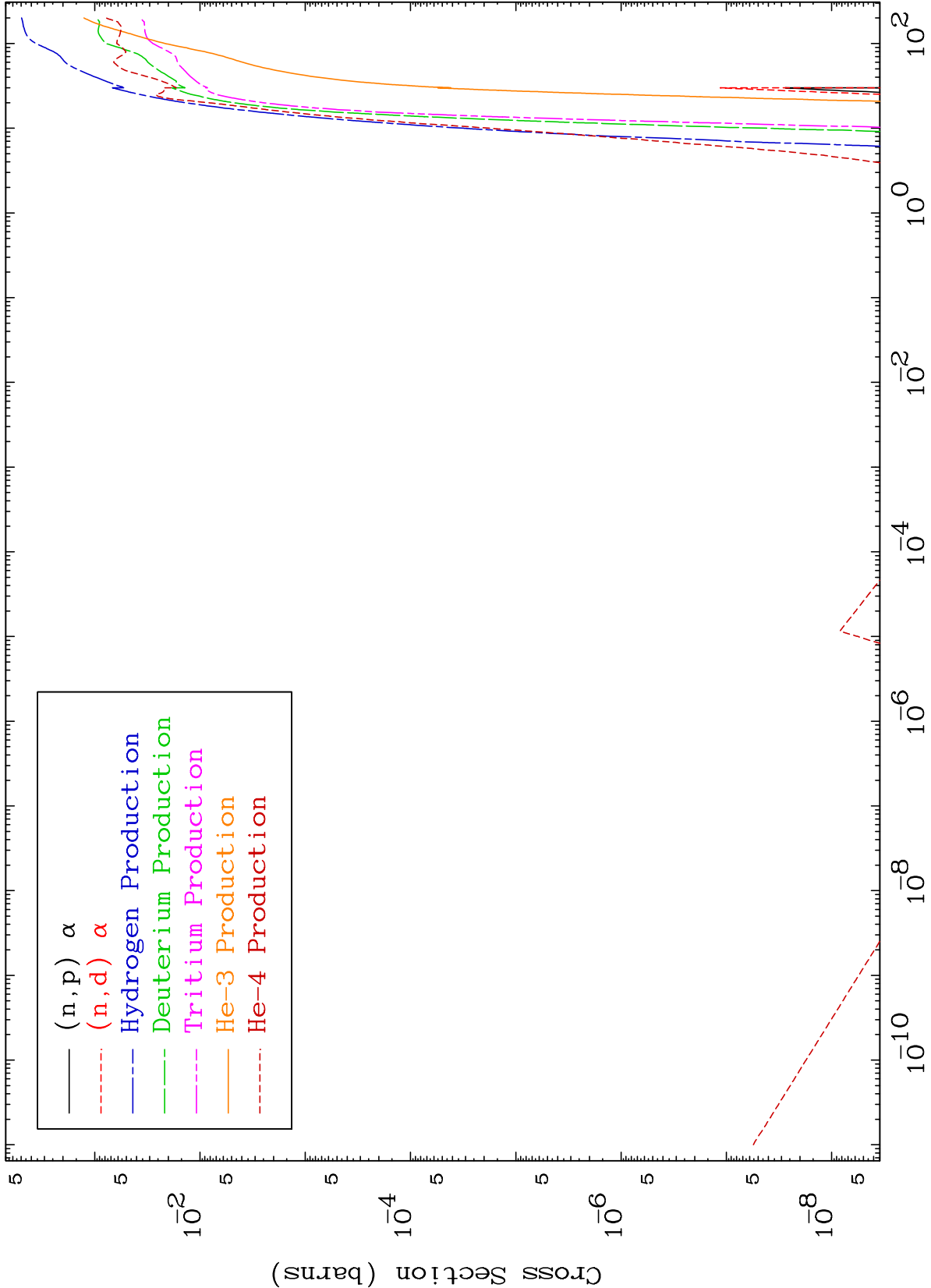




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Neutron Absorption  
293 Kelvin Cross Sections

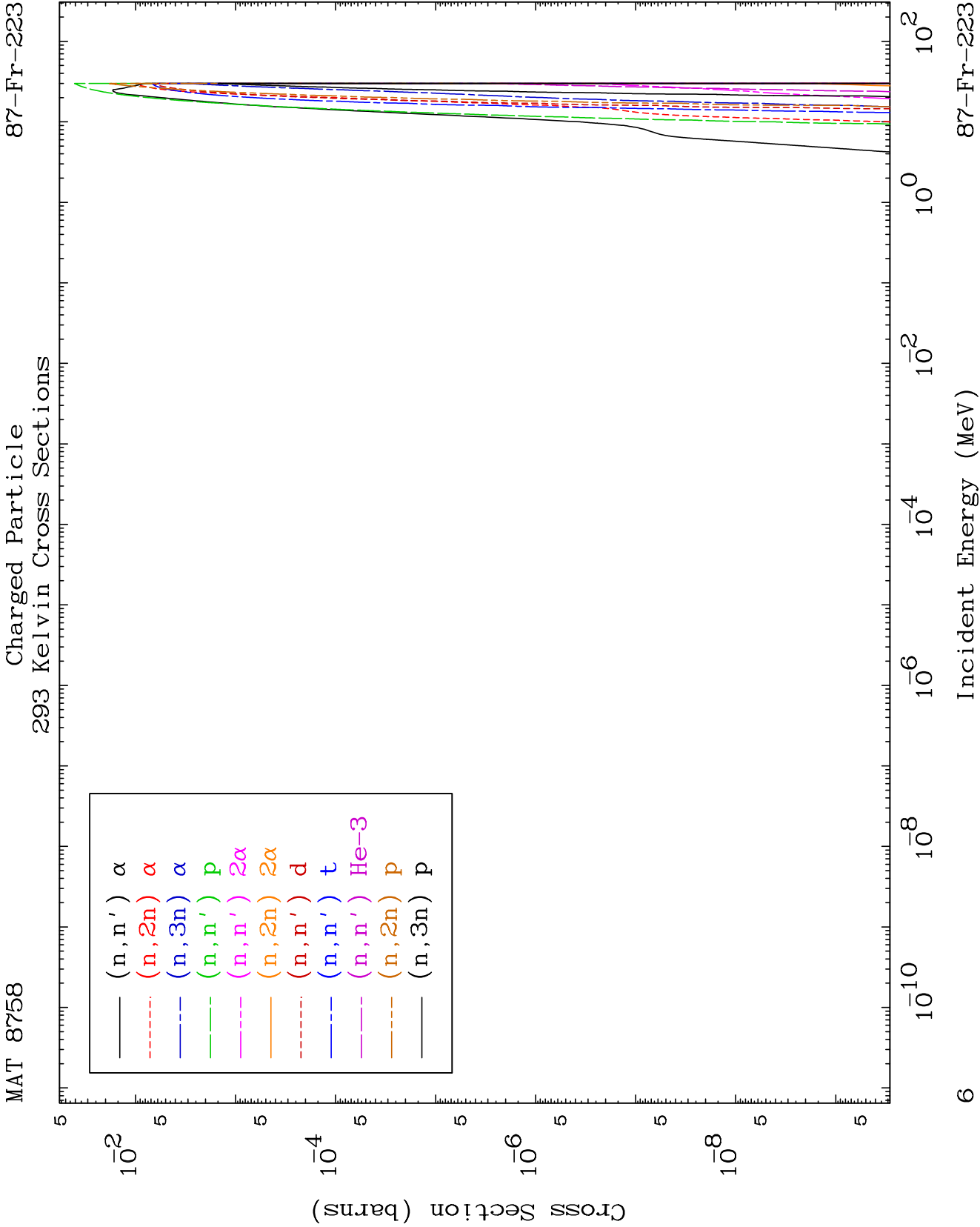
87-Fr-223



5

Incident Energy (MeV)

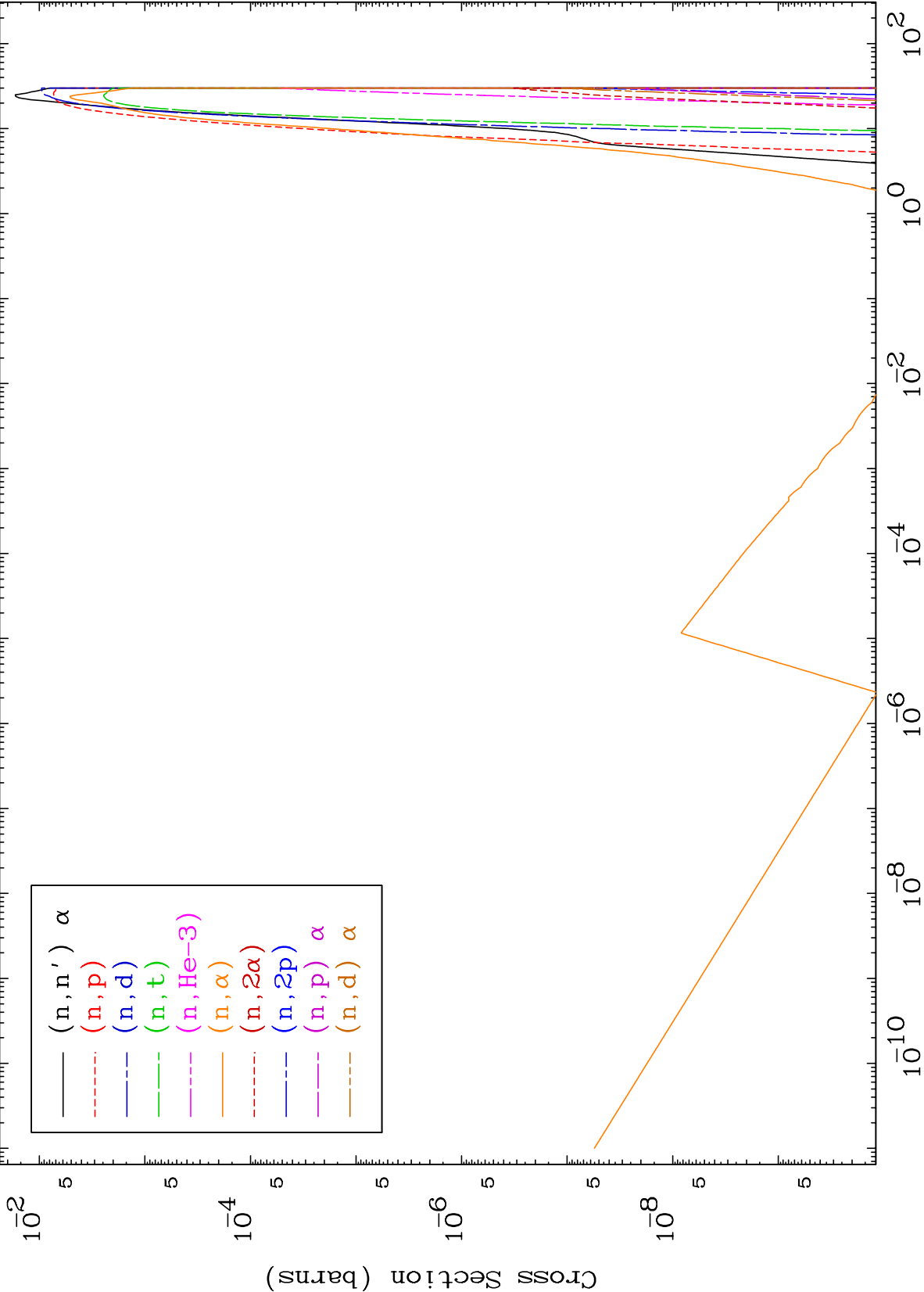
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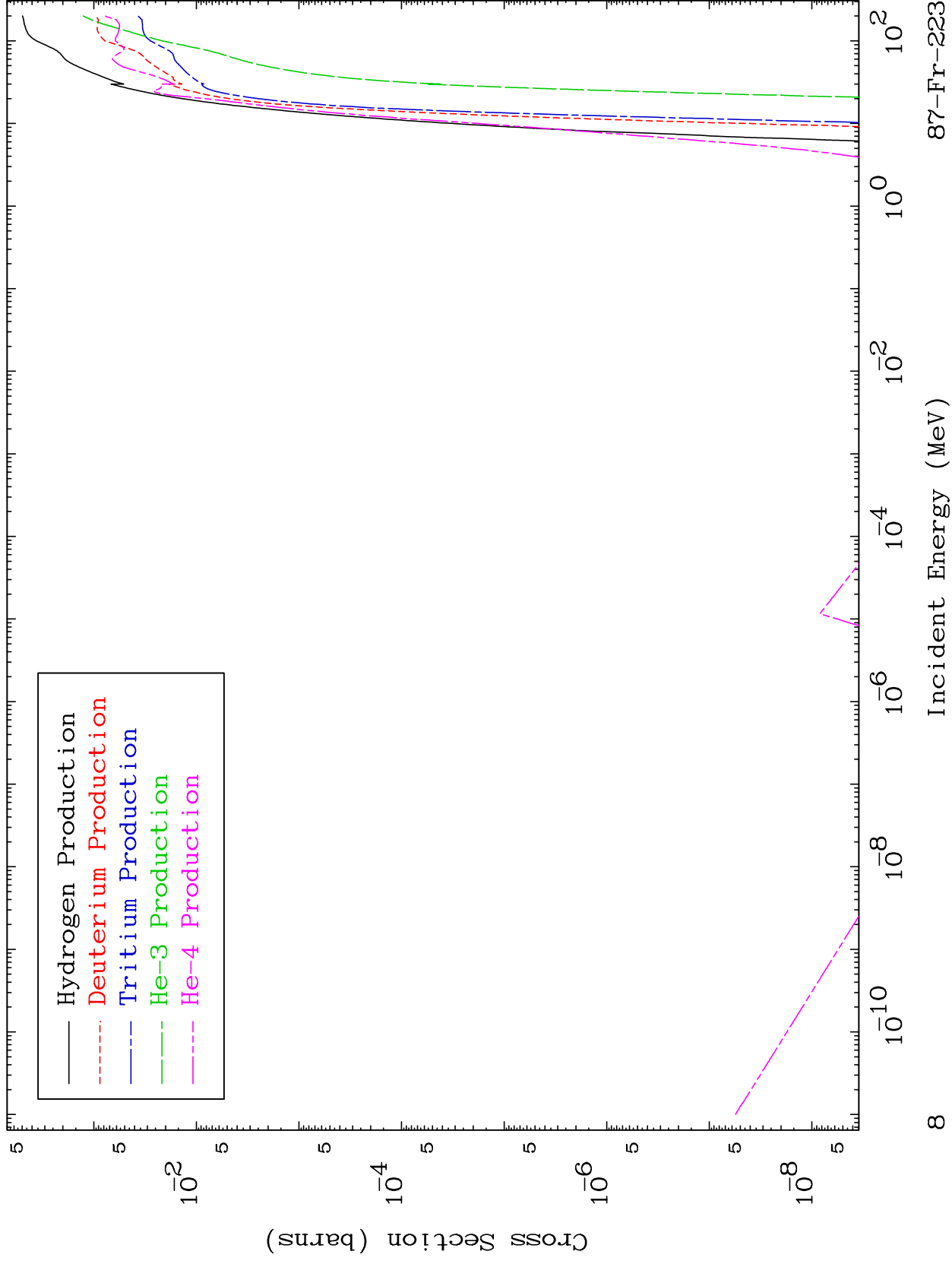
MAT 8758

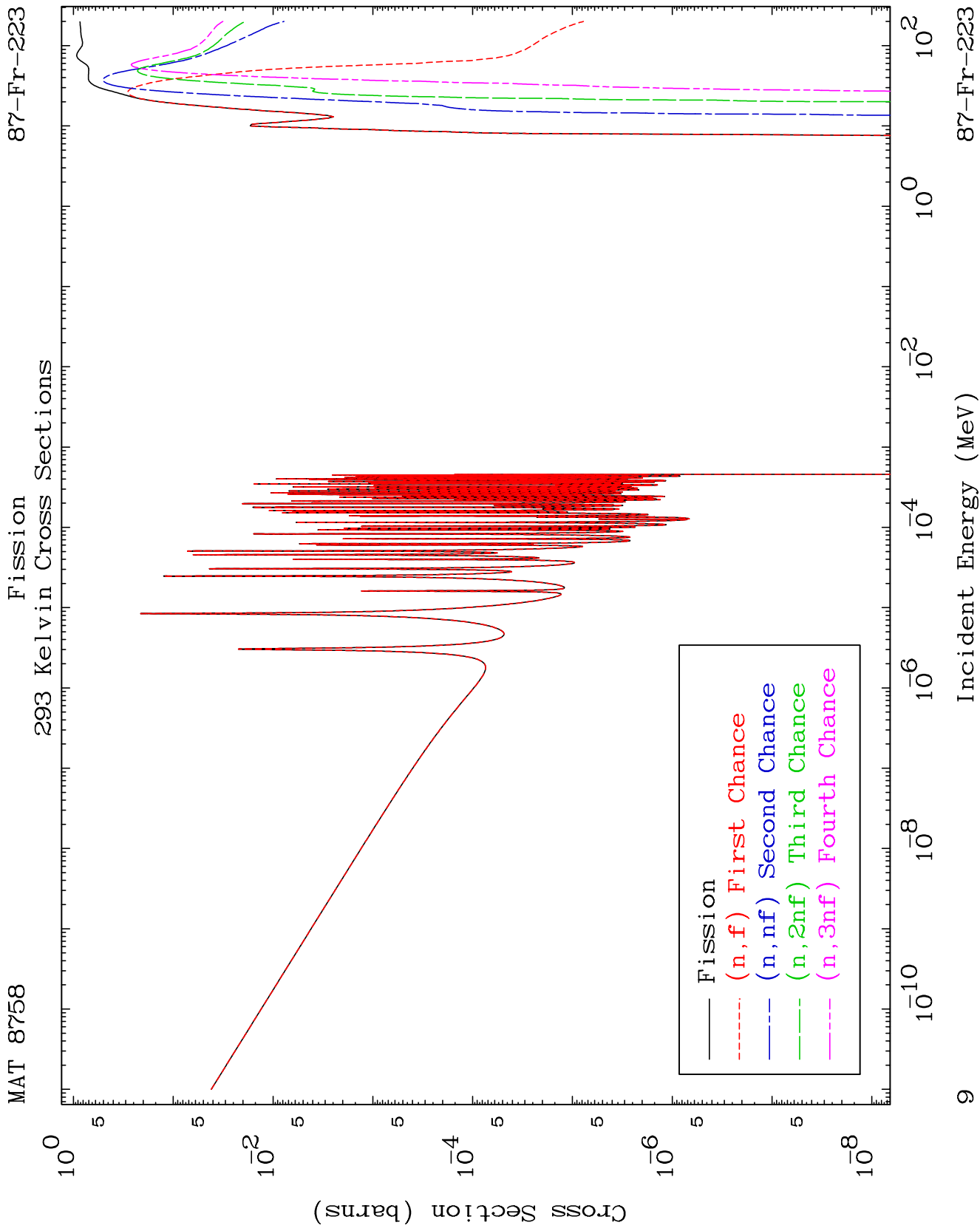
Charged Particle  
293 Kelvin Cross Sections

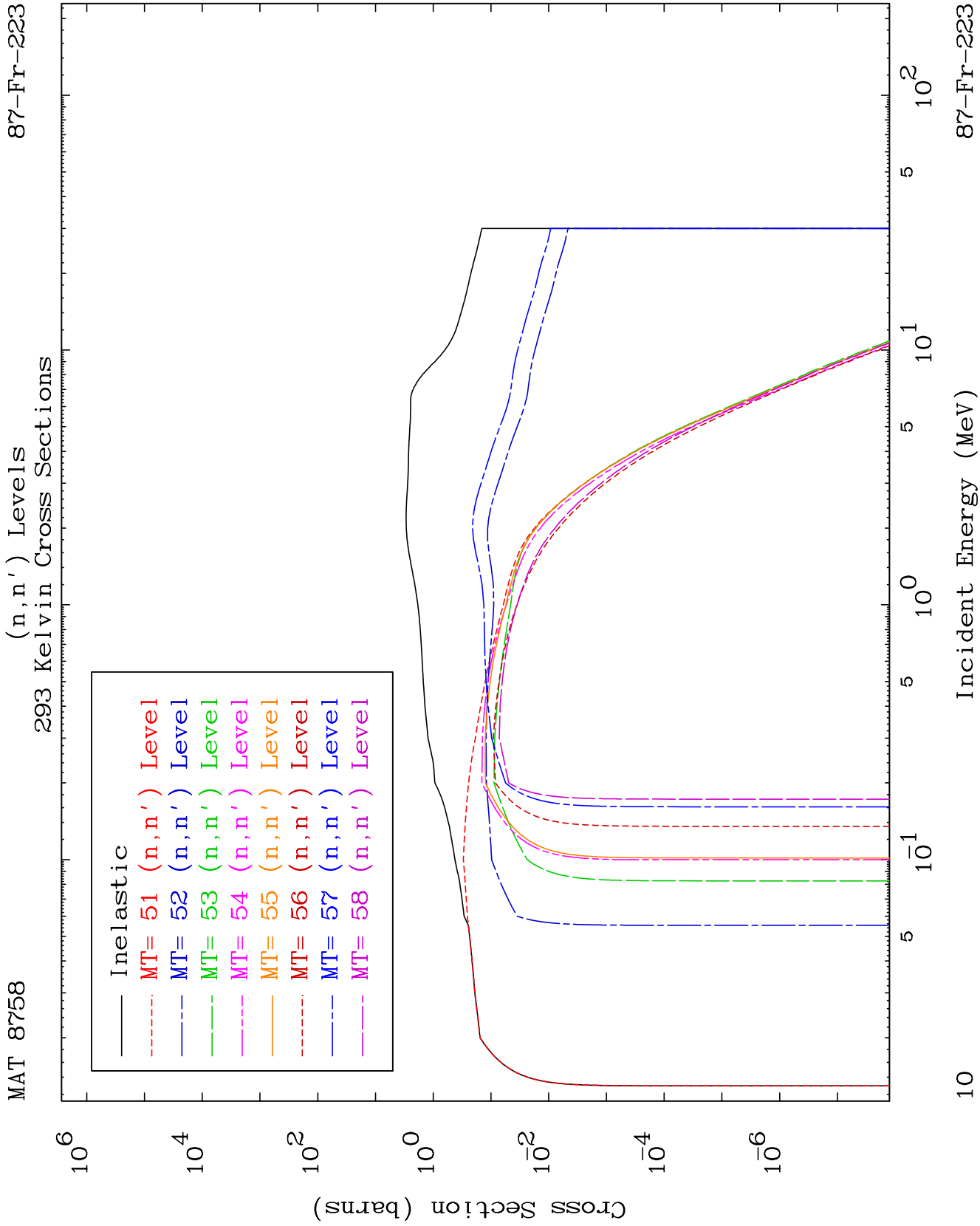
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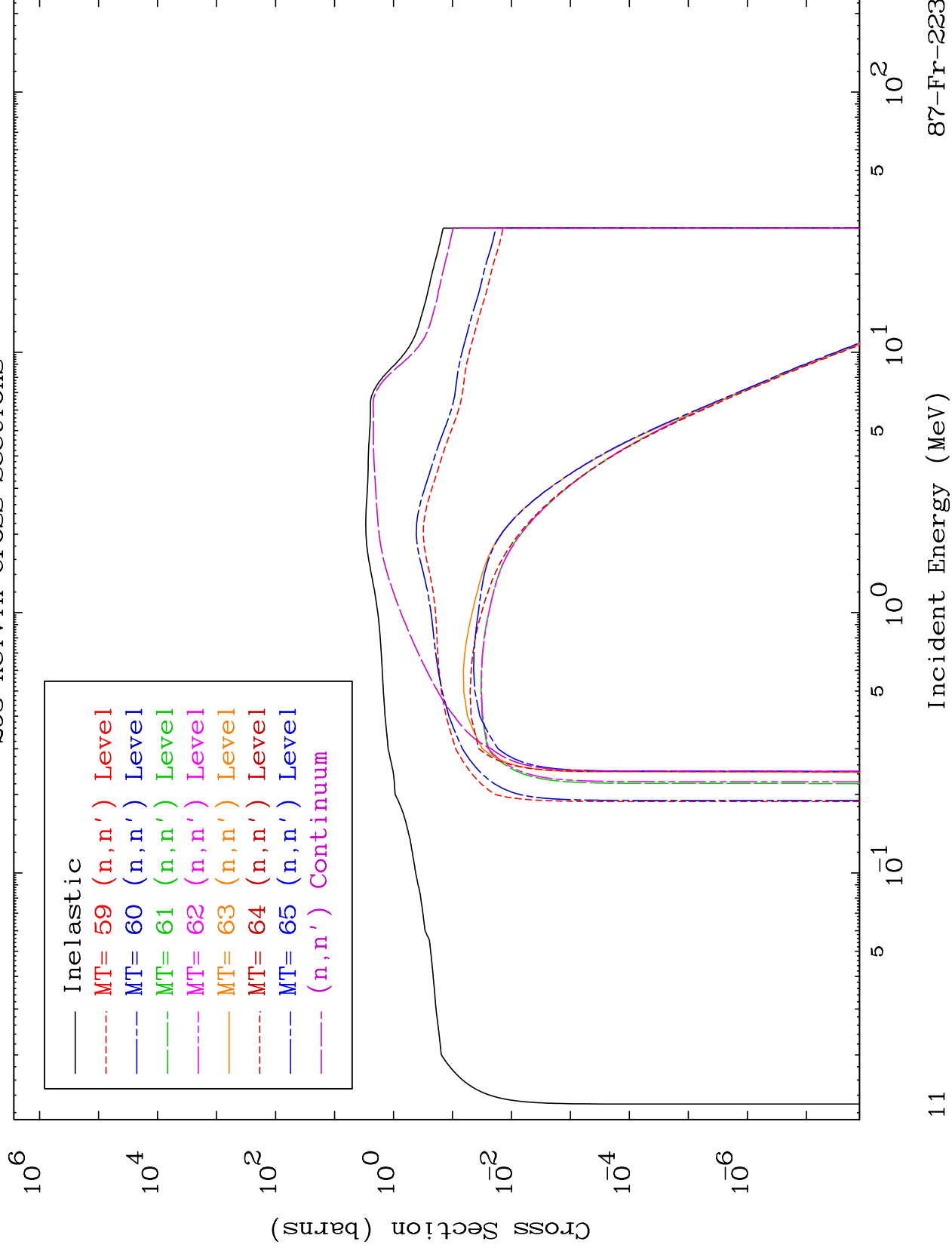


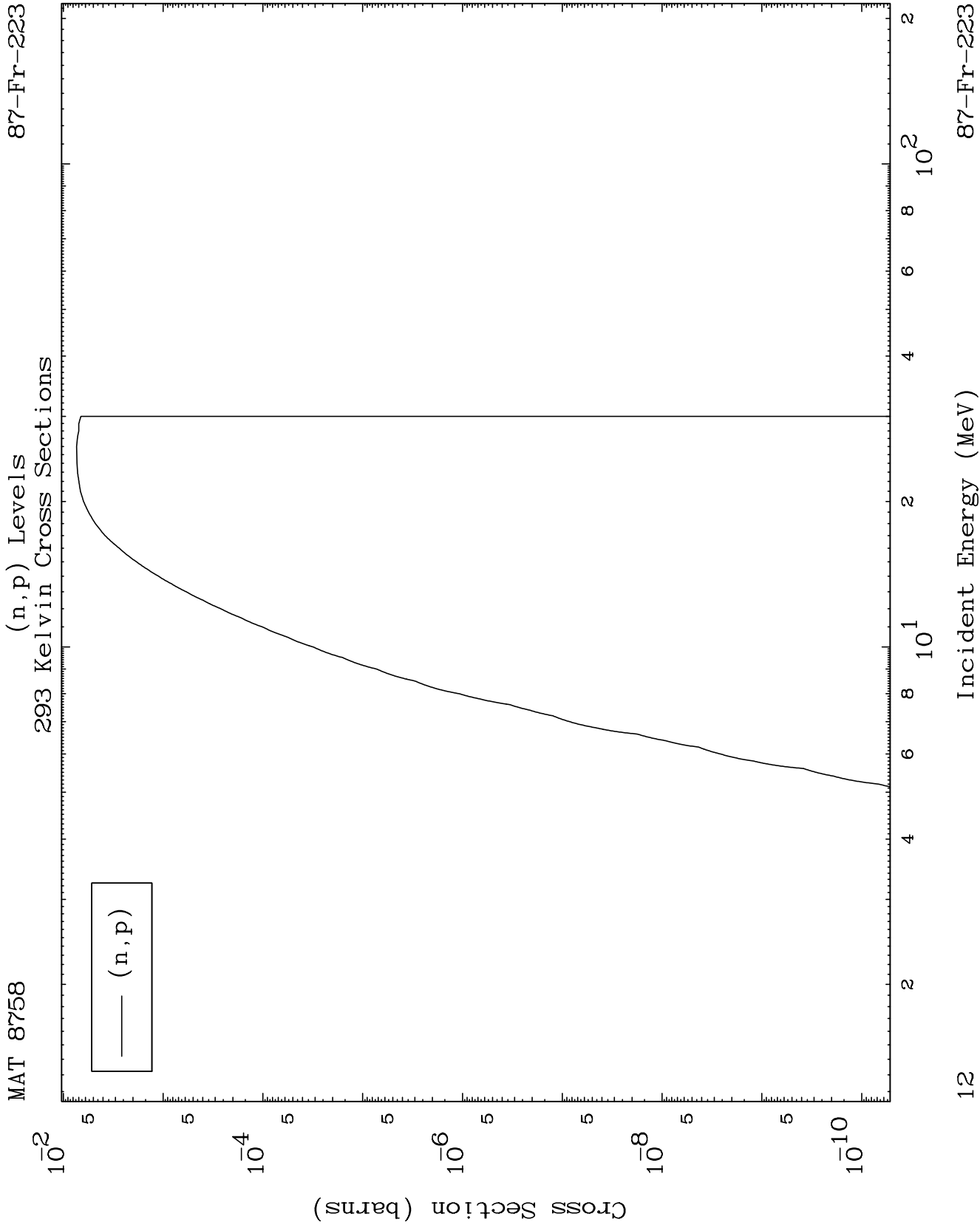








(n,n') Levels  
293 Kelvin Cross Sections

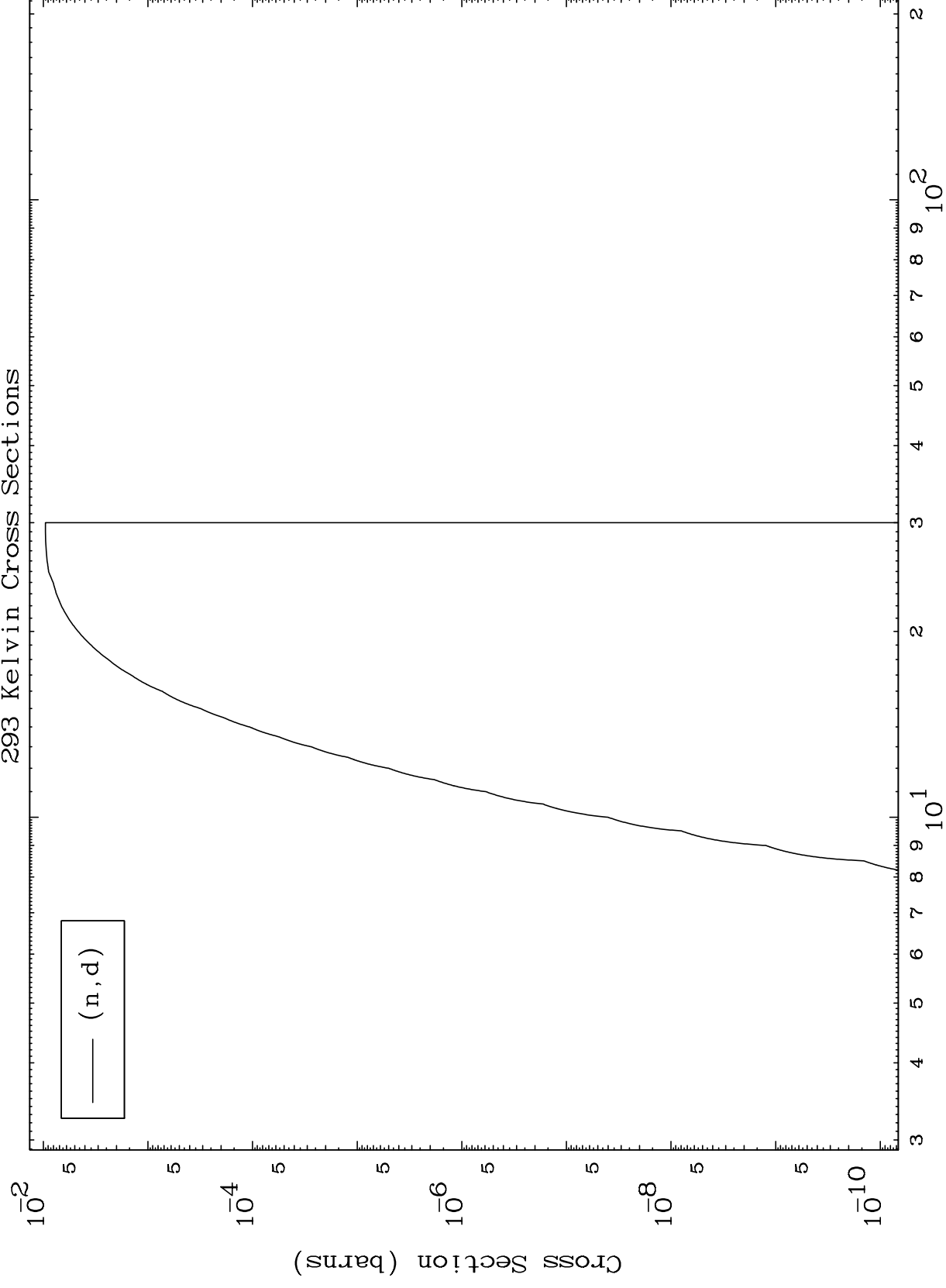


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(n,d) Levels

87-Fr-223

293 Kelvin Cross Sections



13

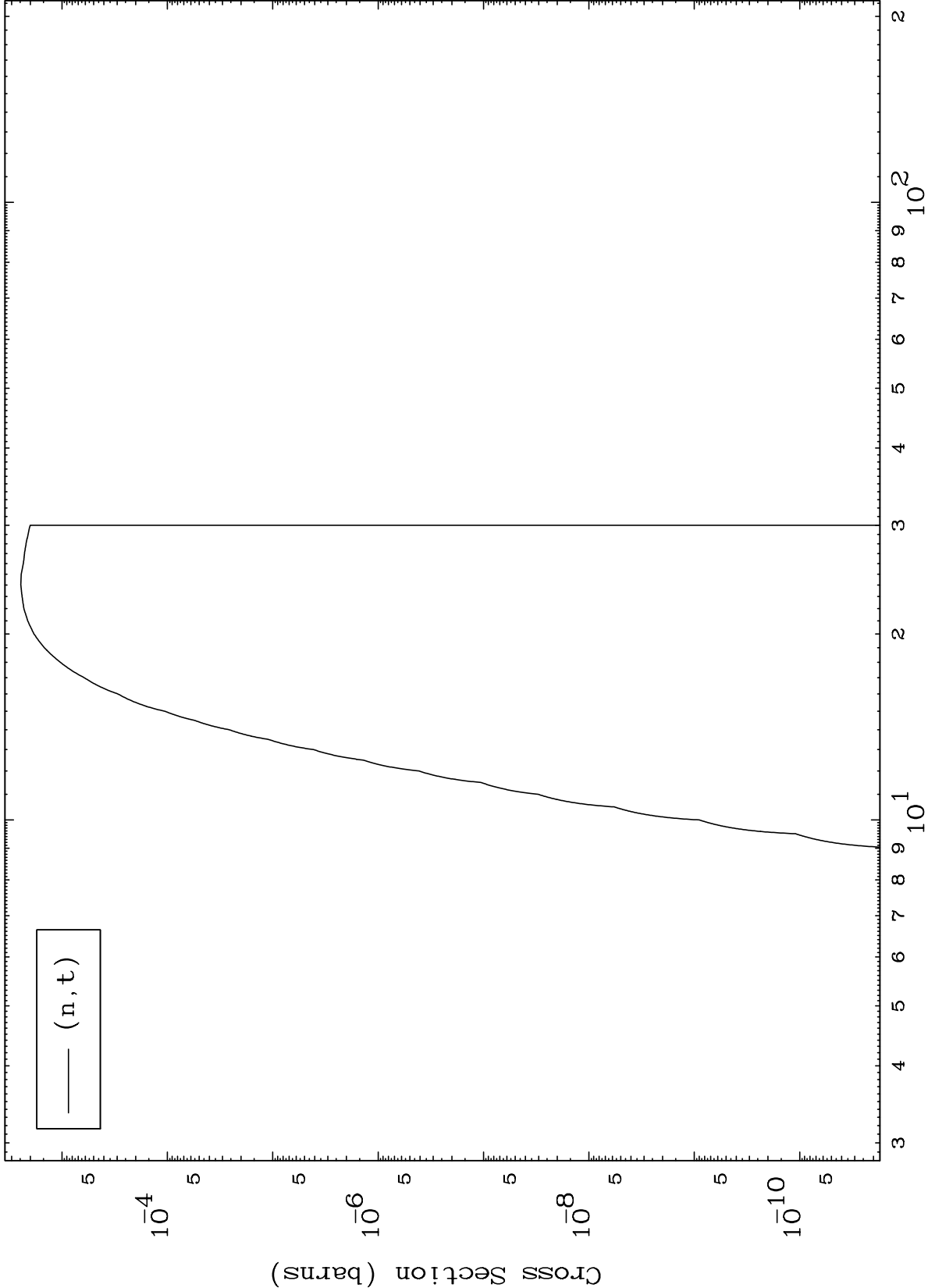
Incident Energy (MeV)

87-Fr-223

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(n,t) Levels  
293 Kelvin Cross Sections

87-Fr-223



14

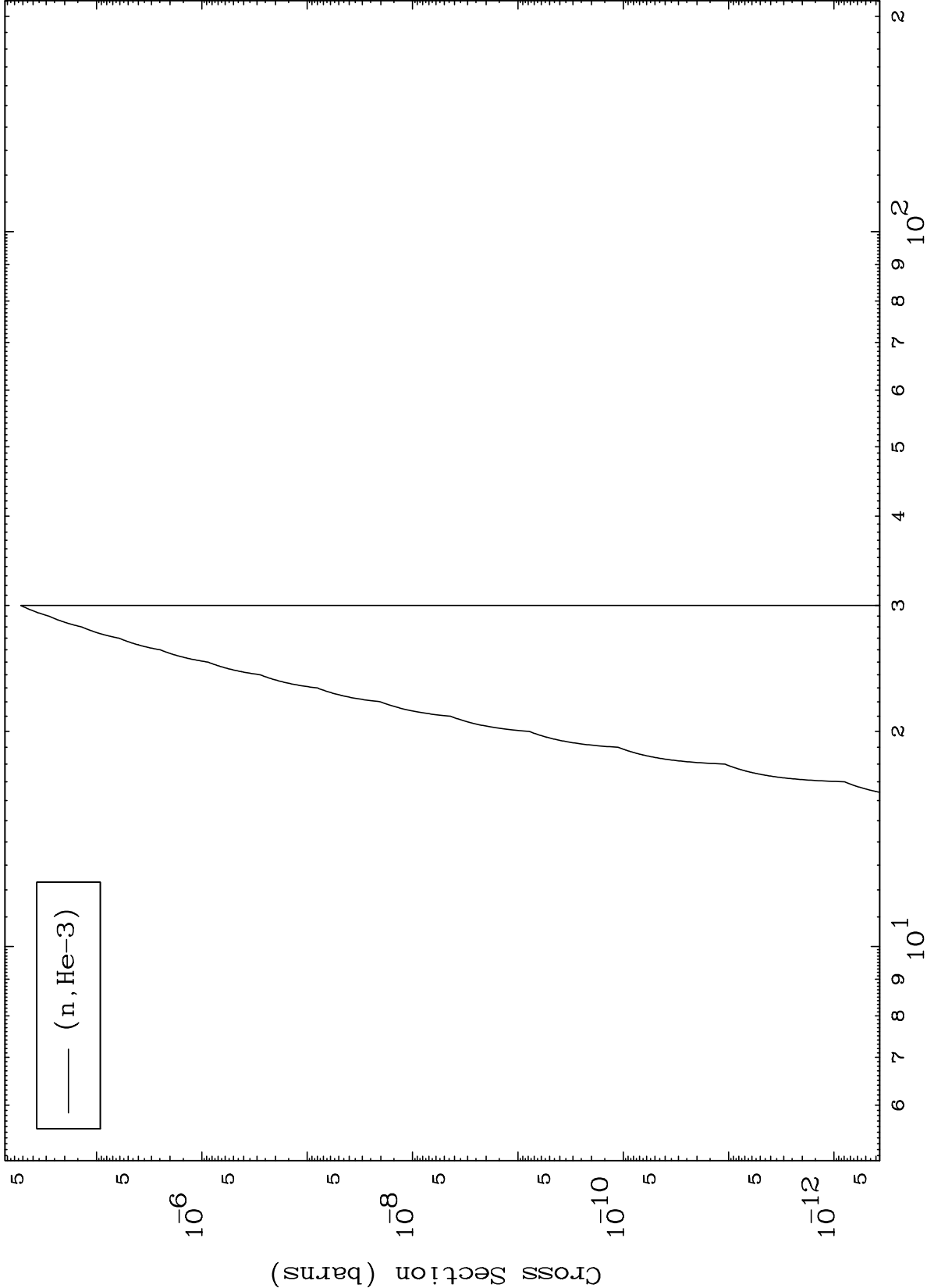
Incident Energy (MeV)

87-Fr-223

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(n,He3) Levels  
293 Kelvin Cross Sections

87-Fr-223

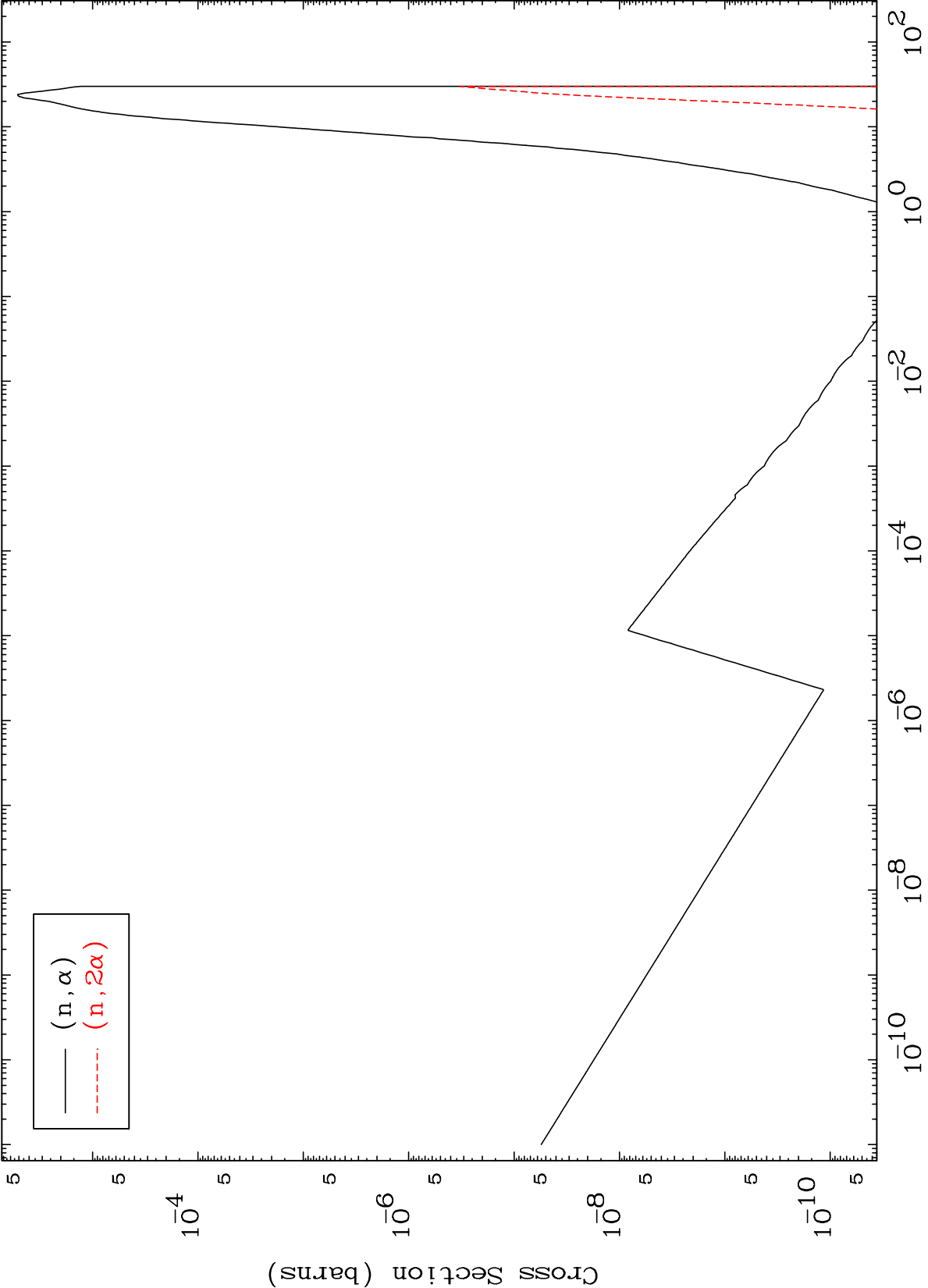


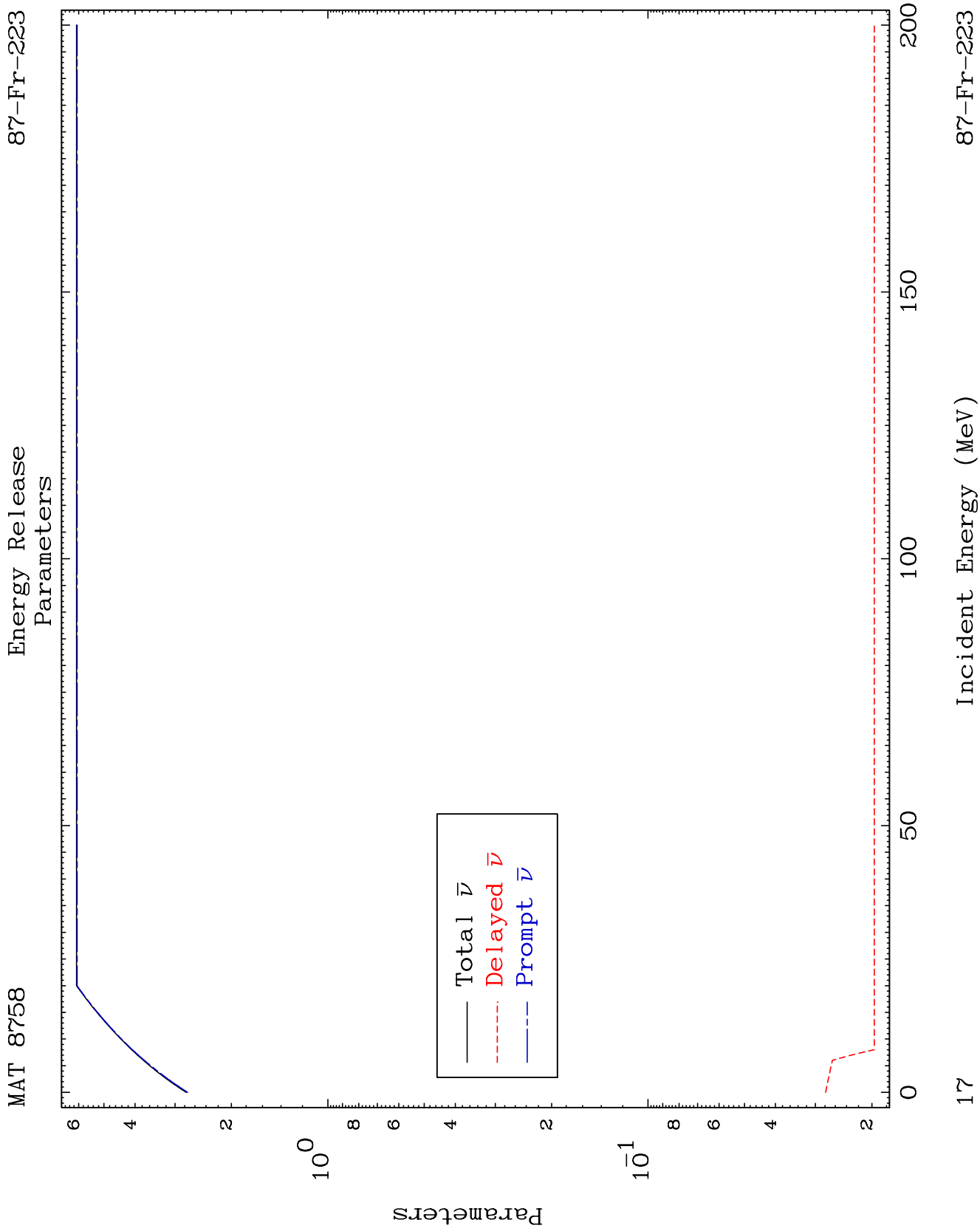
15

Incident Energy (MeV)

87-Fr-223



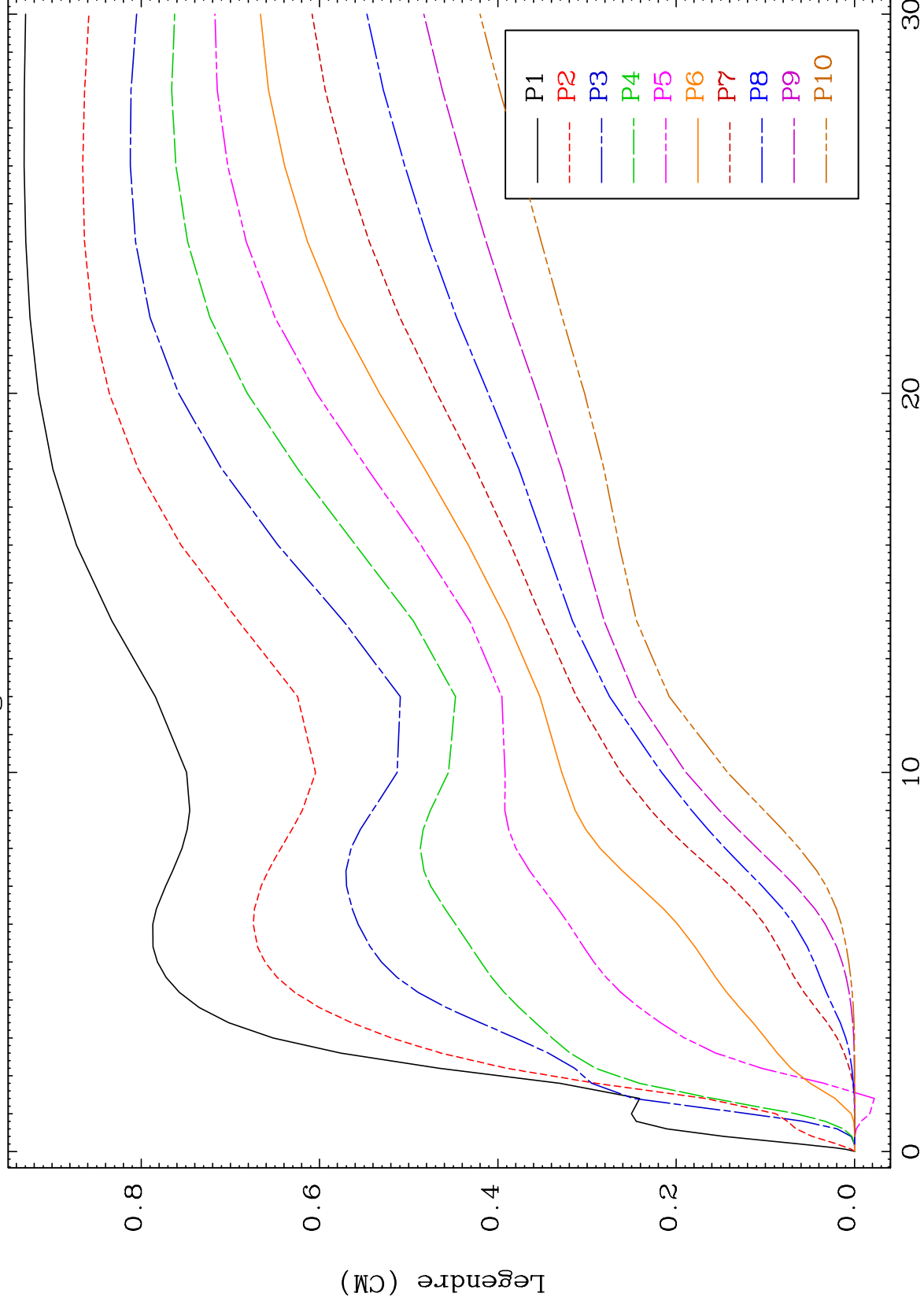




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# Elastic Legendre Coefficients



18

Incident Energy (MeV)

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30

20

10

0

P1

23

P3

P4

59

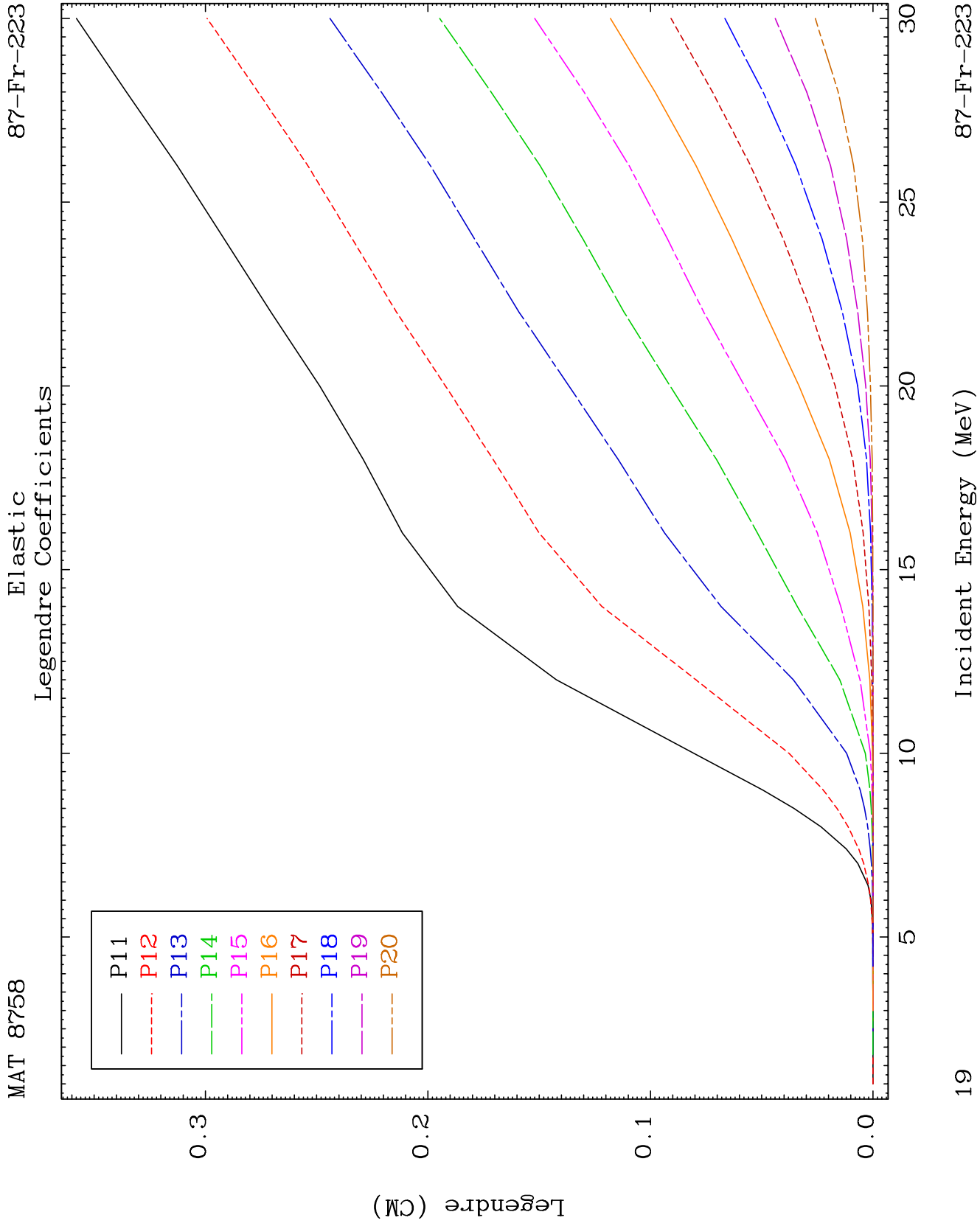
99

29

89

2

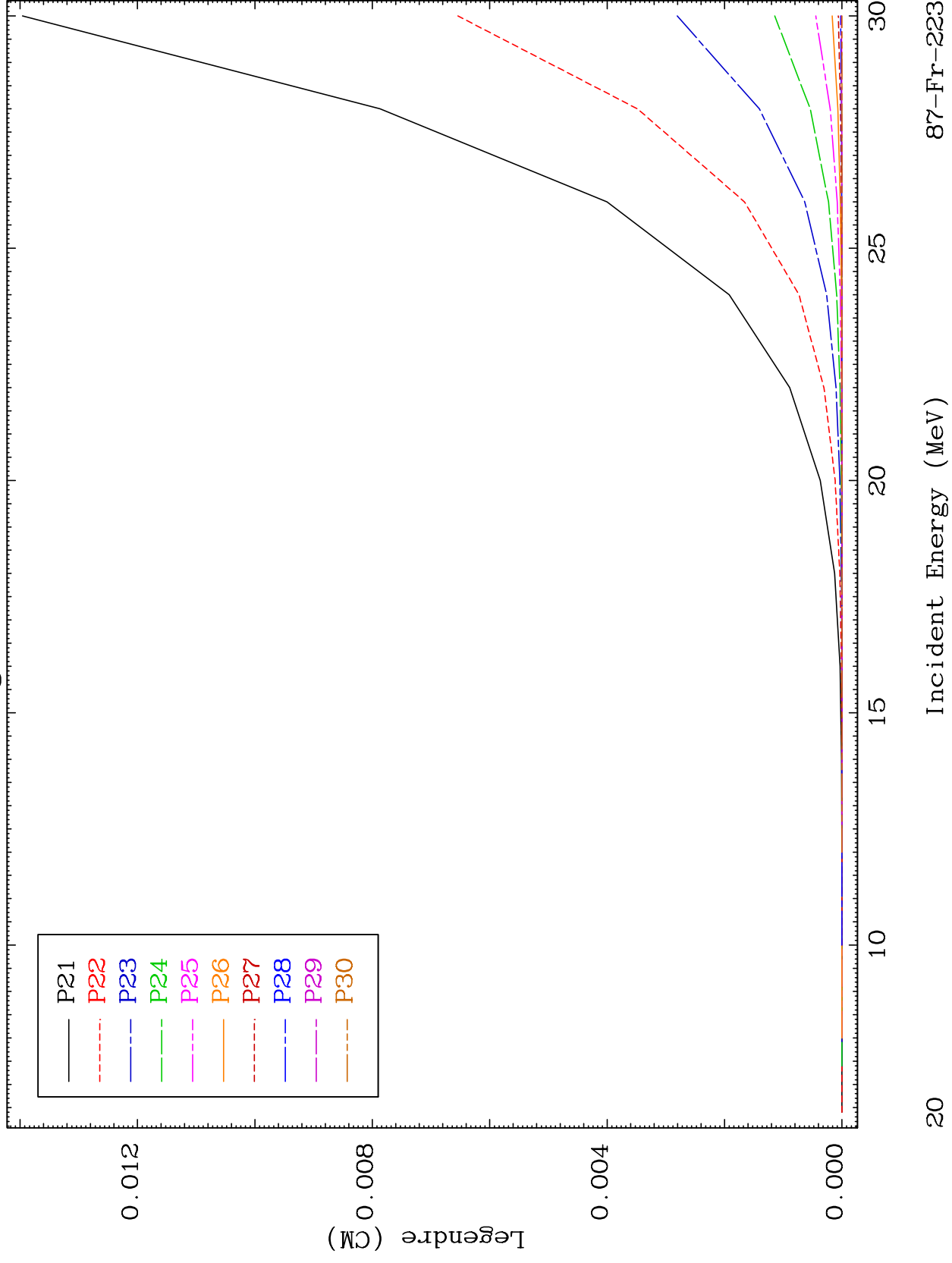
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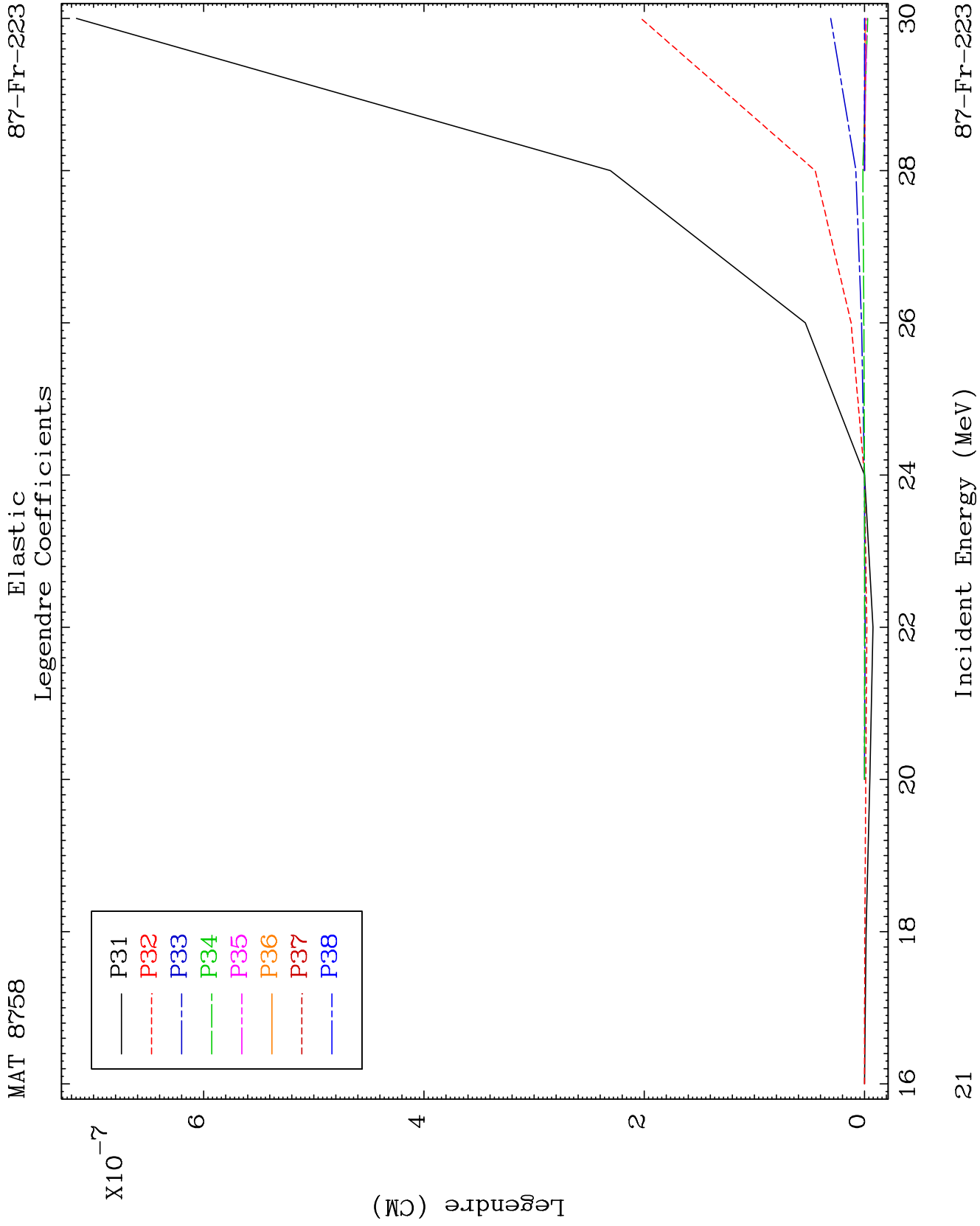


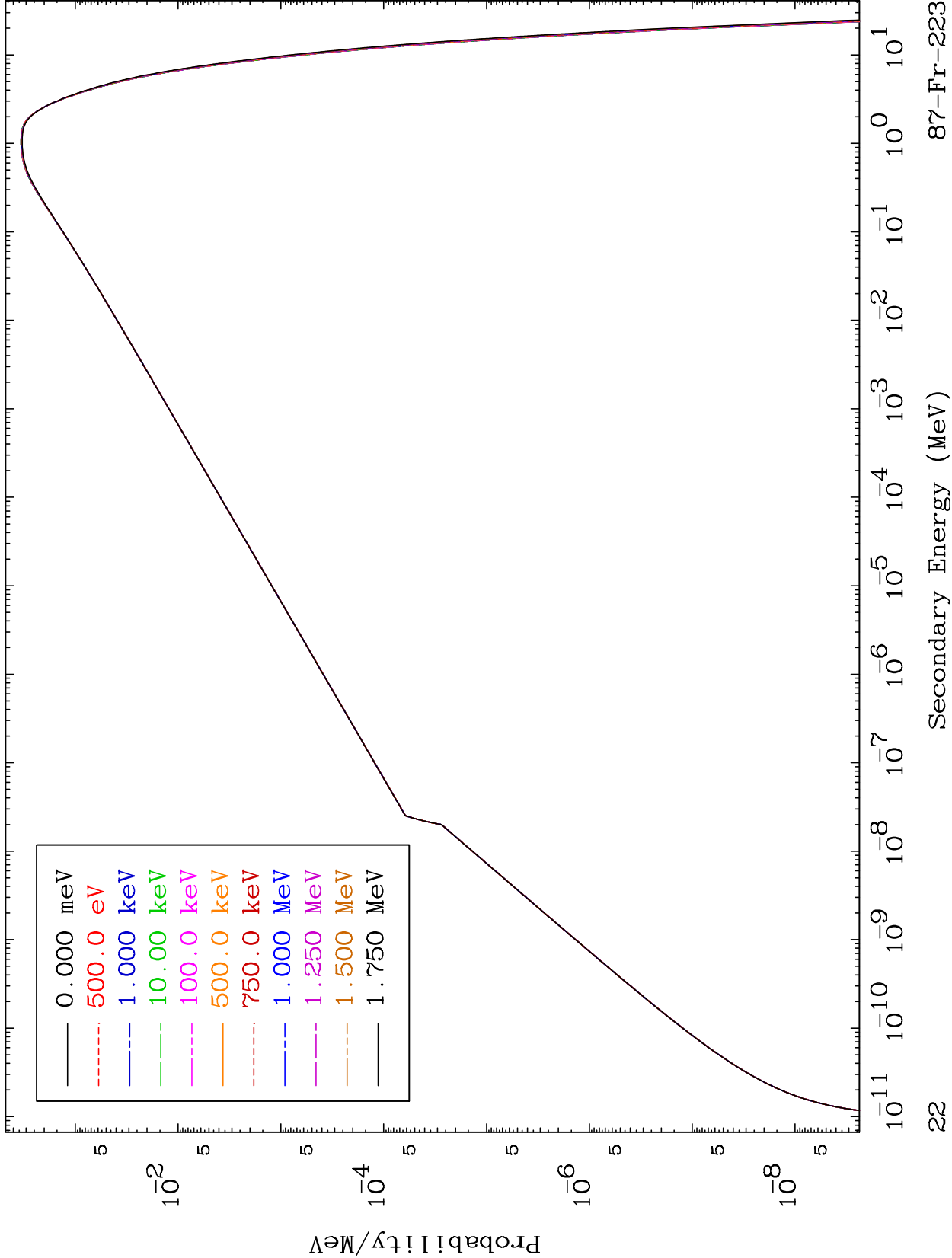
MAT 8758

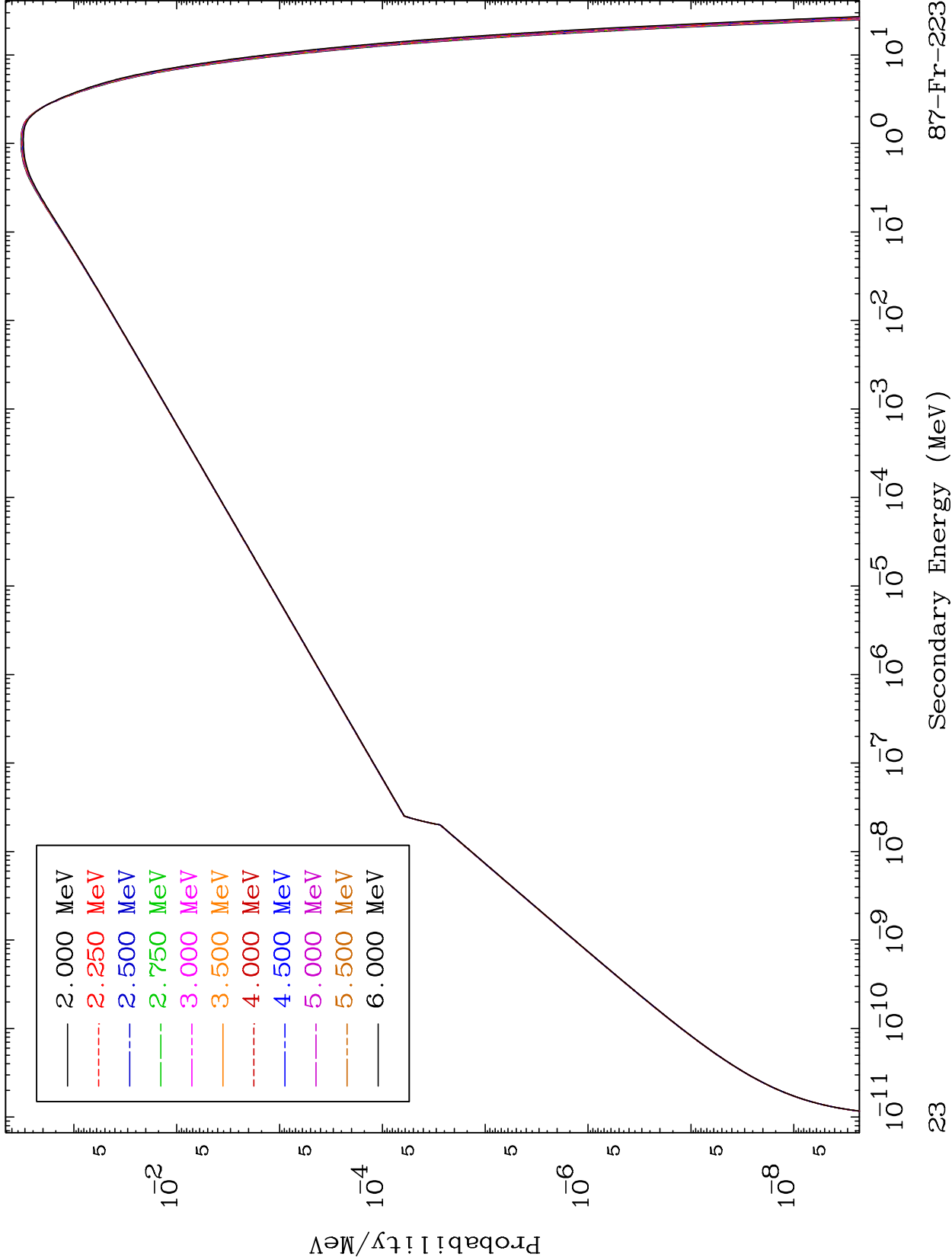
## Elastic Legendre Coefficients

87-Fr-223

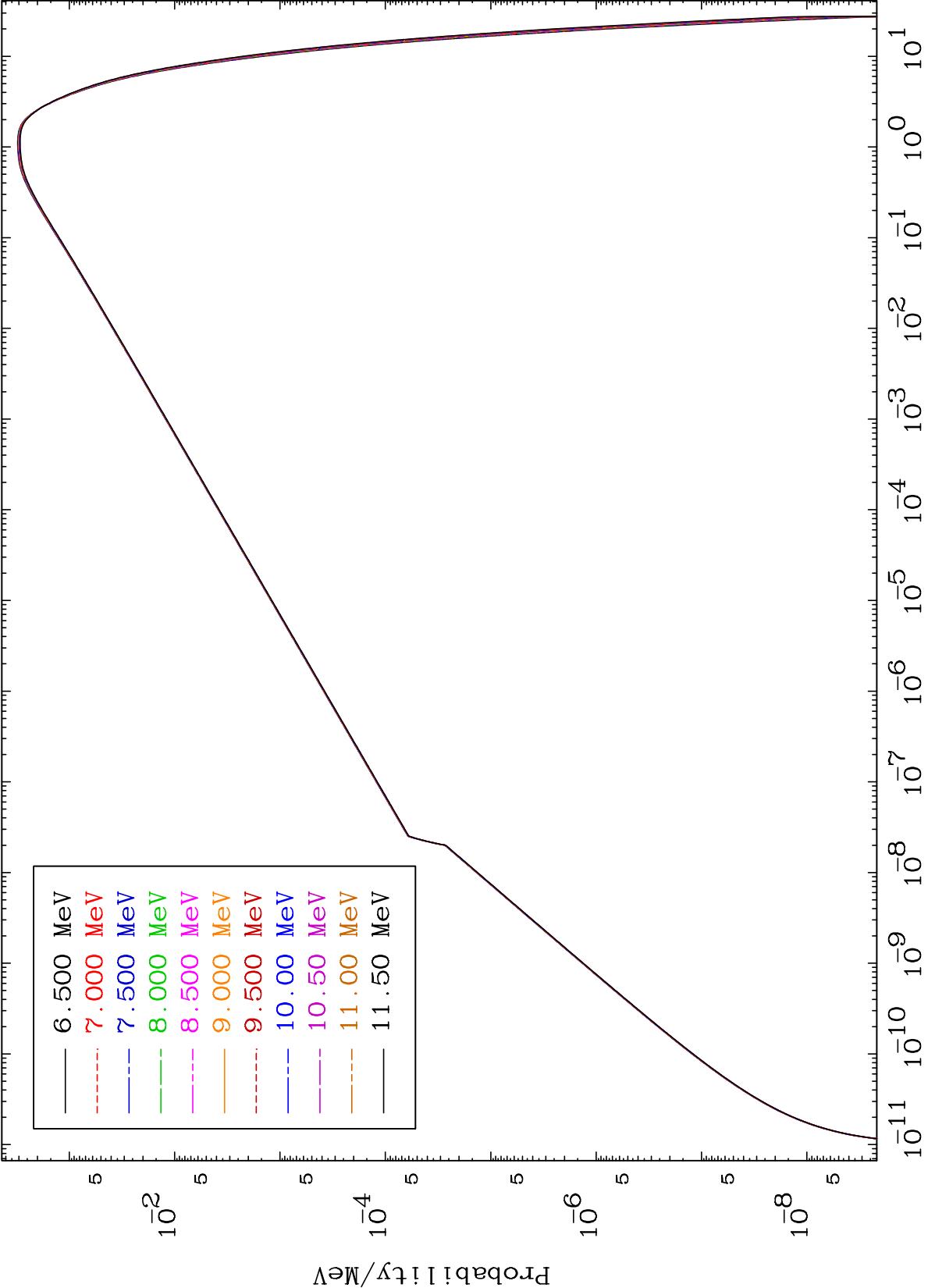








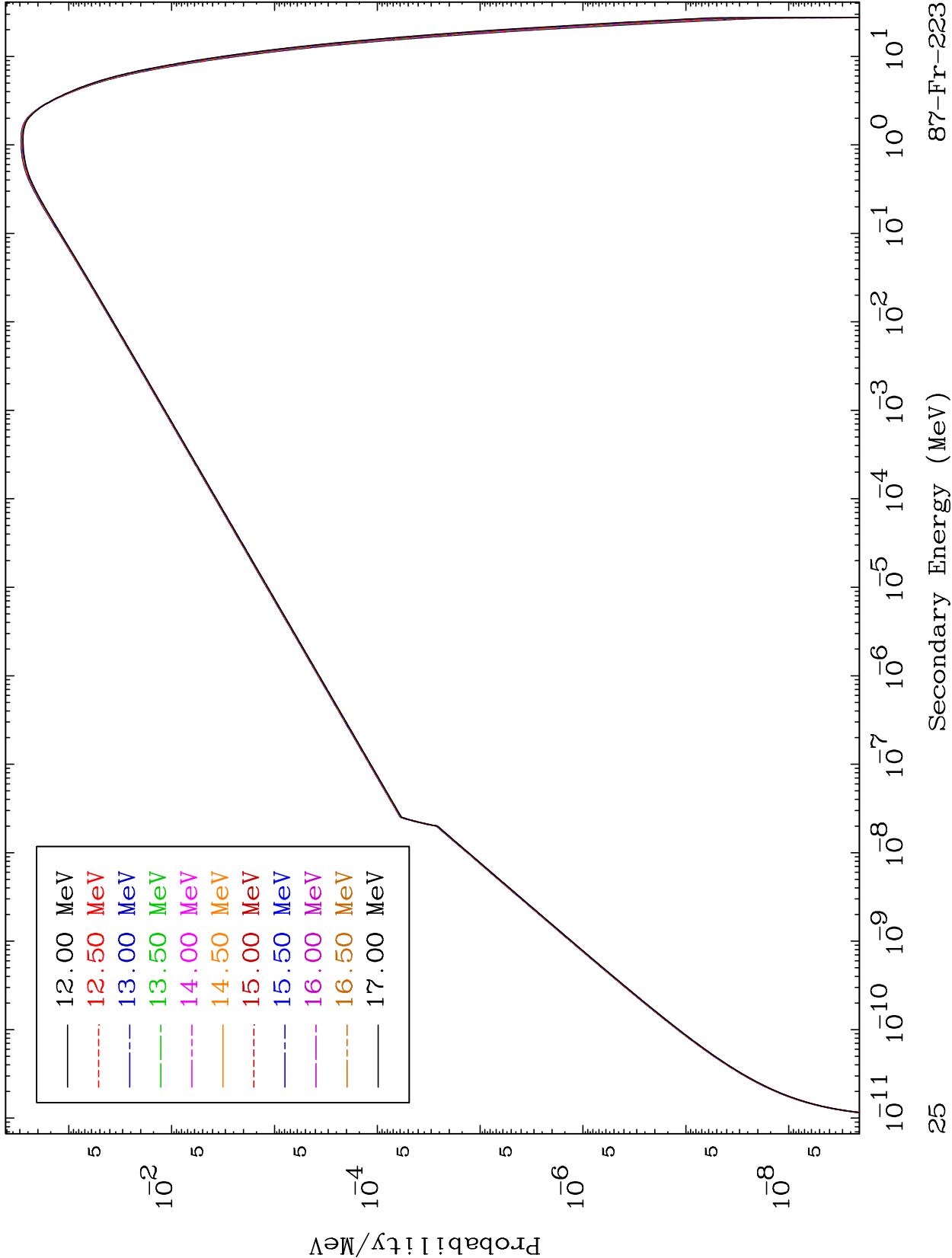


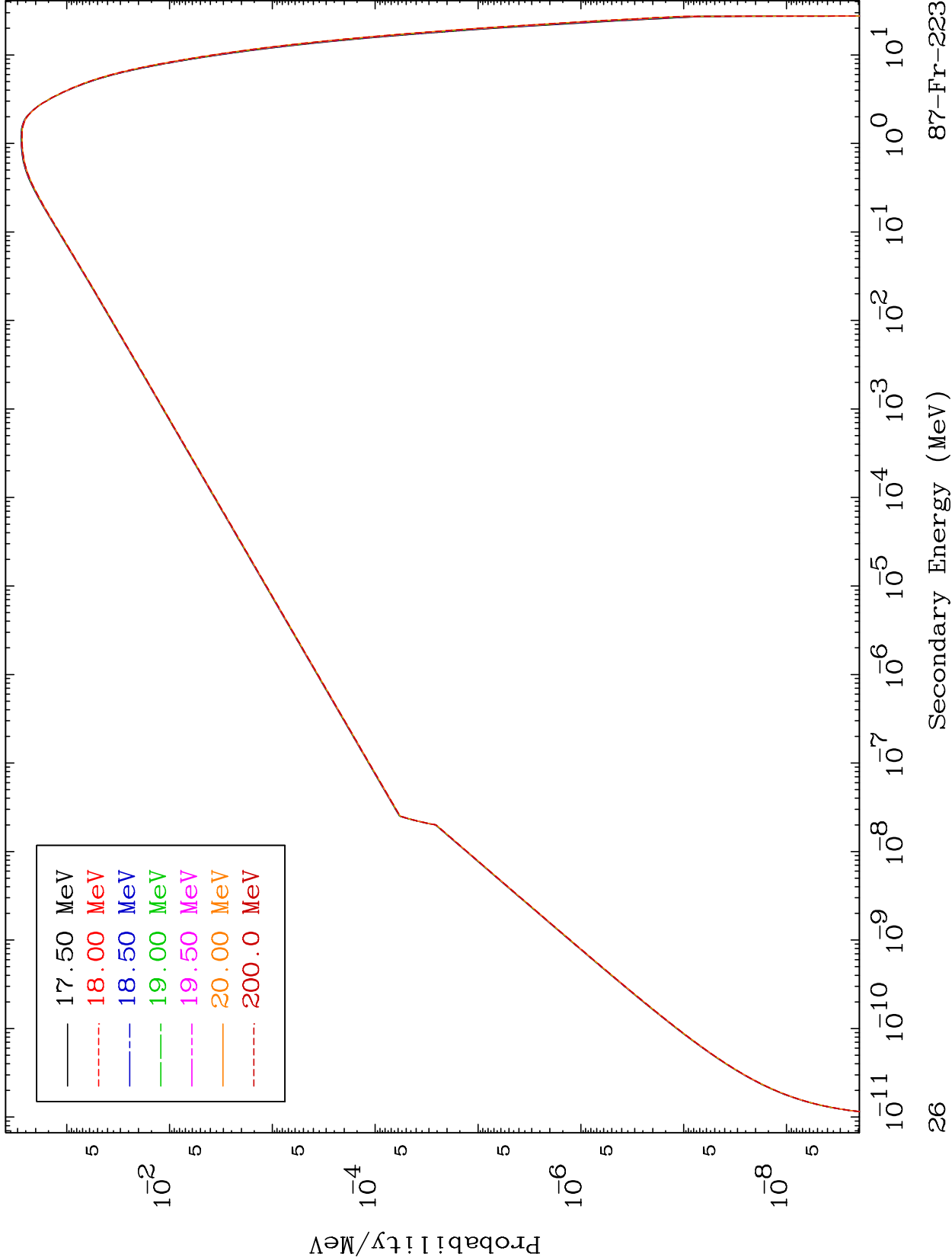


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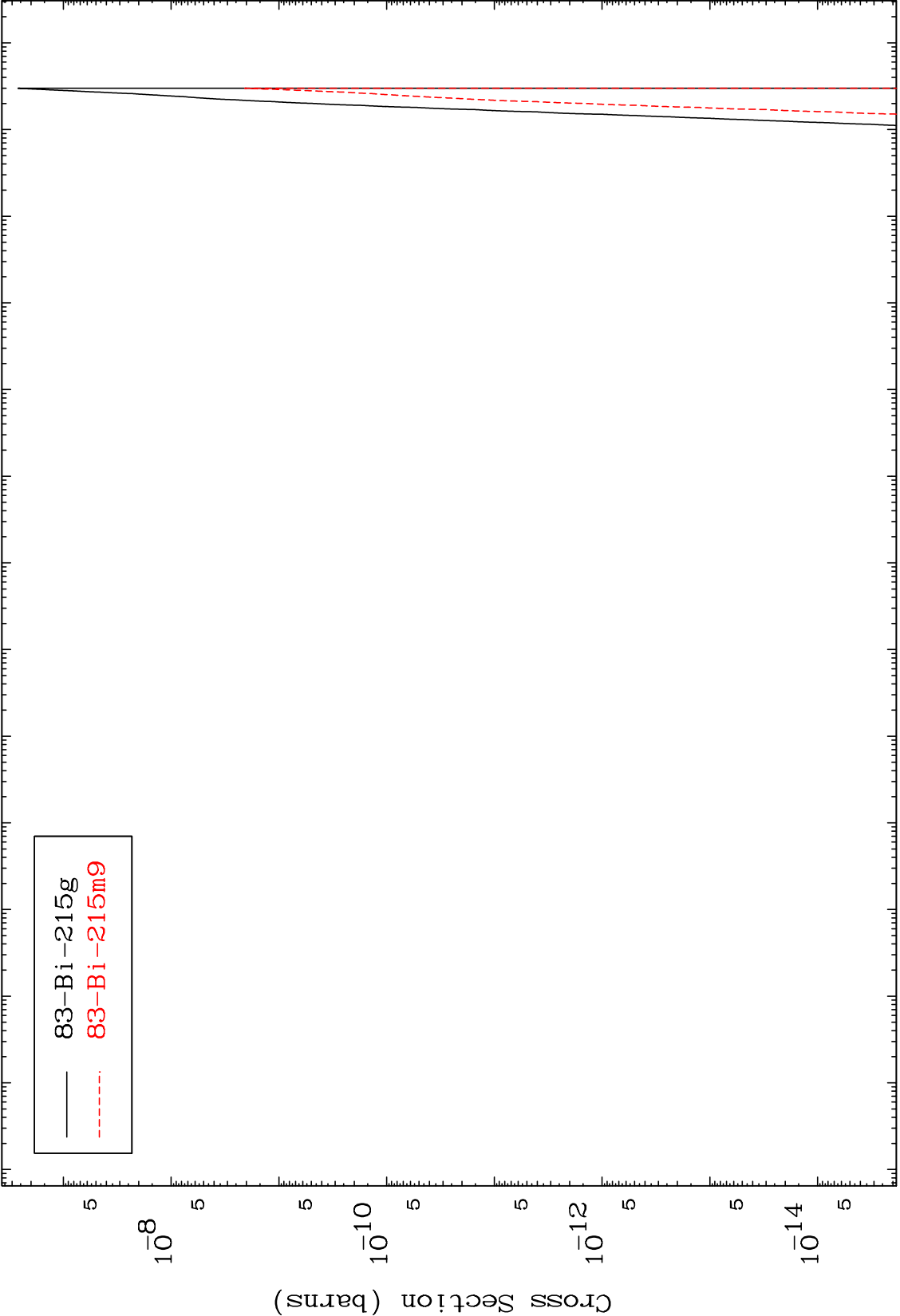
Fission  
Energy Distributions

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Radionuclide Production Cross Section

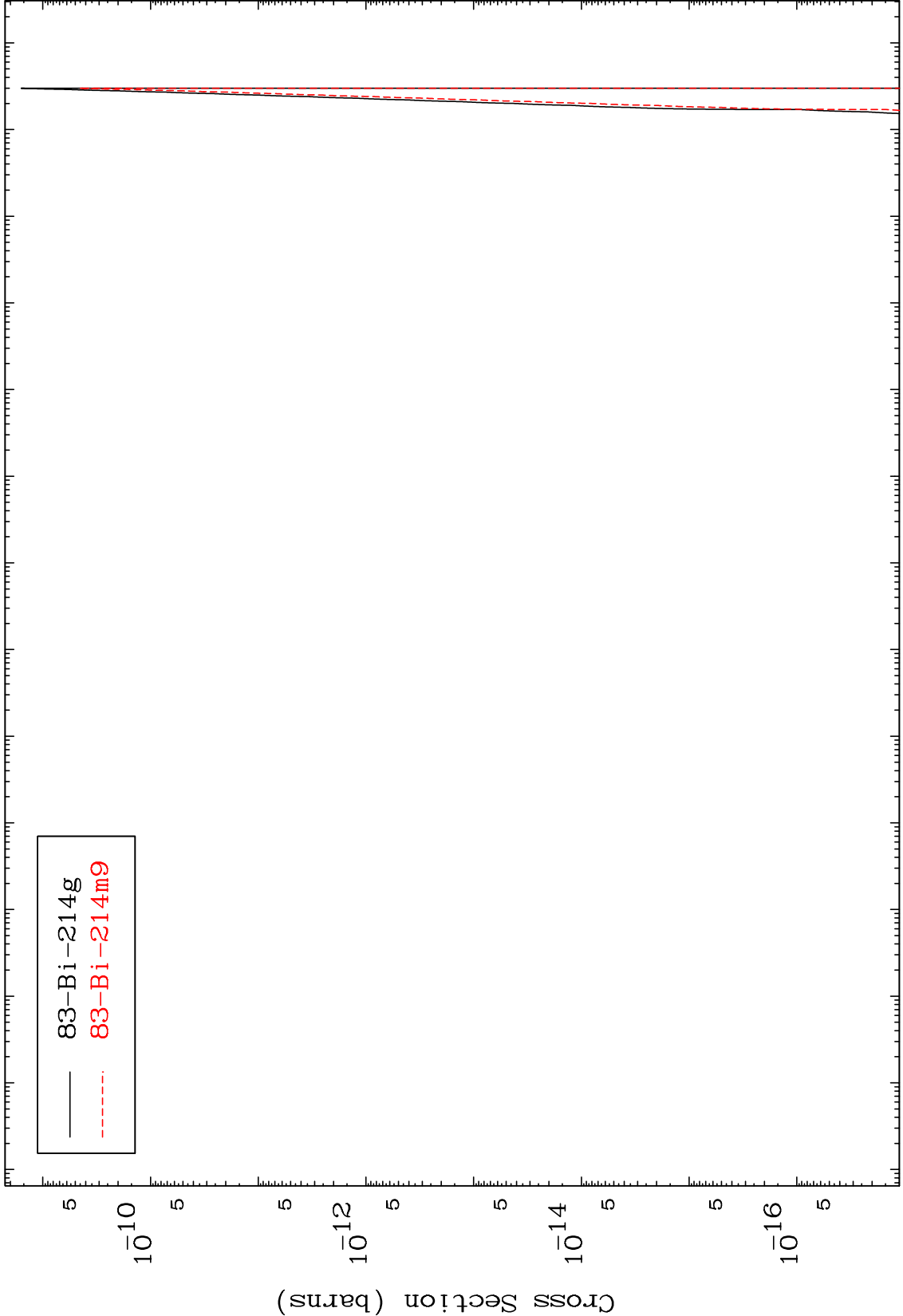


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(n,2n) 2 $\alpha$

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Radionuclide Production Cross Section



28

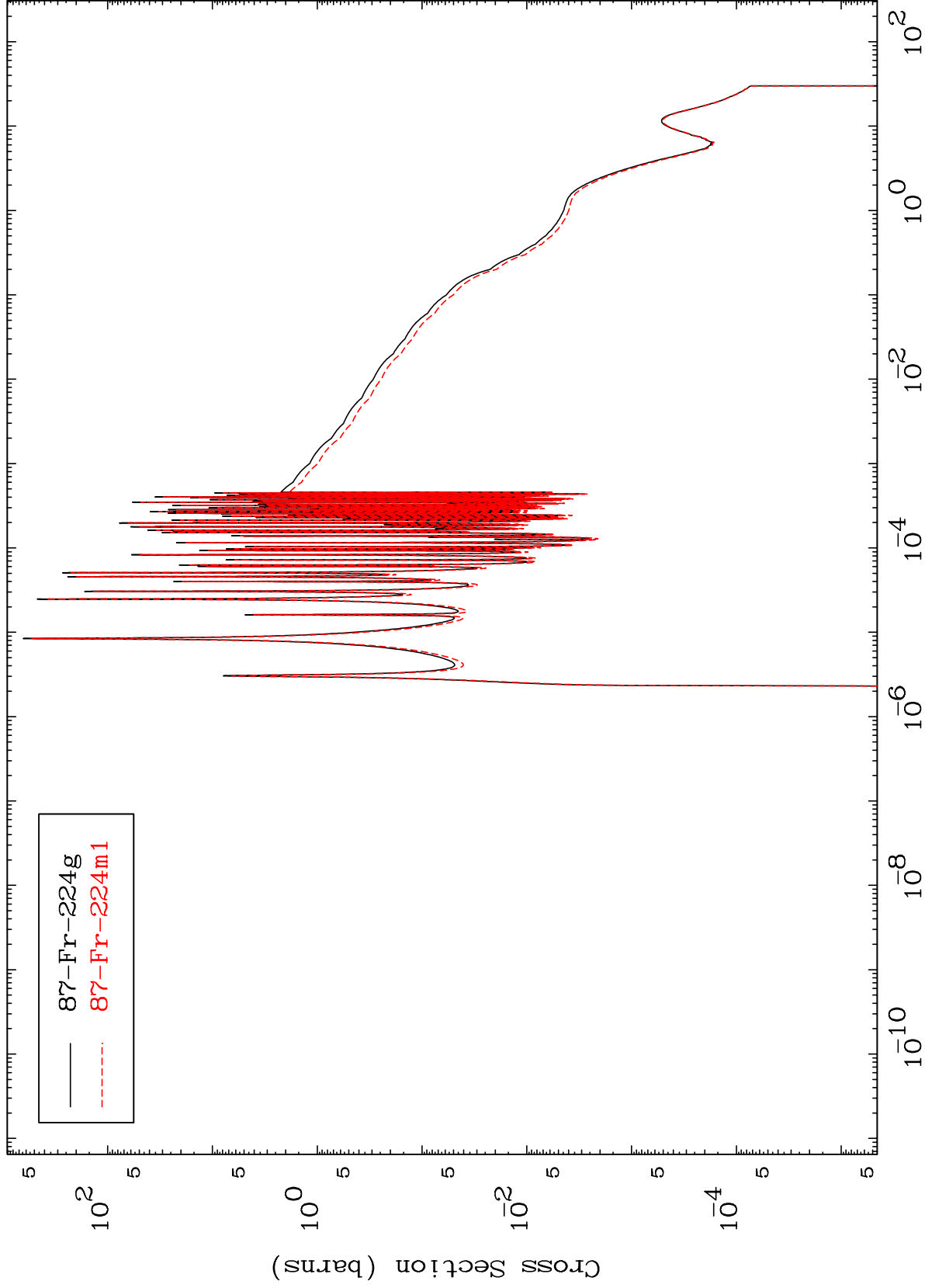
Incident Energy (MeV)

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87-Fr-223

Radionuclide Production Cross Section



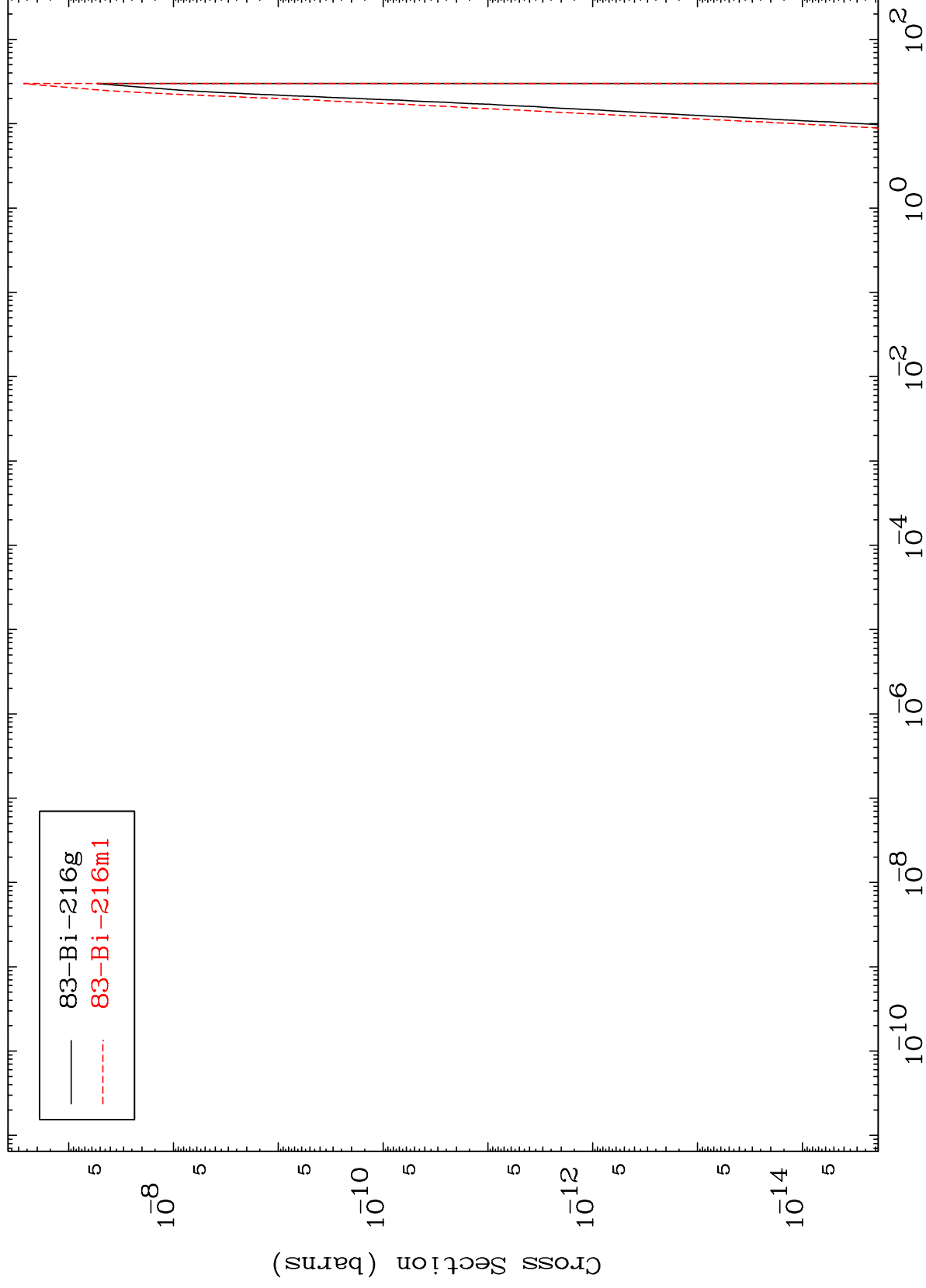
29

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(n,2α)  
Radionuclide Production Cross Section



30

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