

Program EVALPLOT
(Version 2021-1)

by

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(Present Contact Information)

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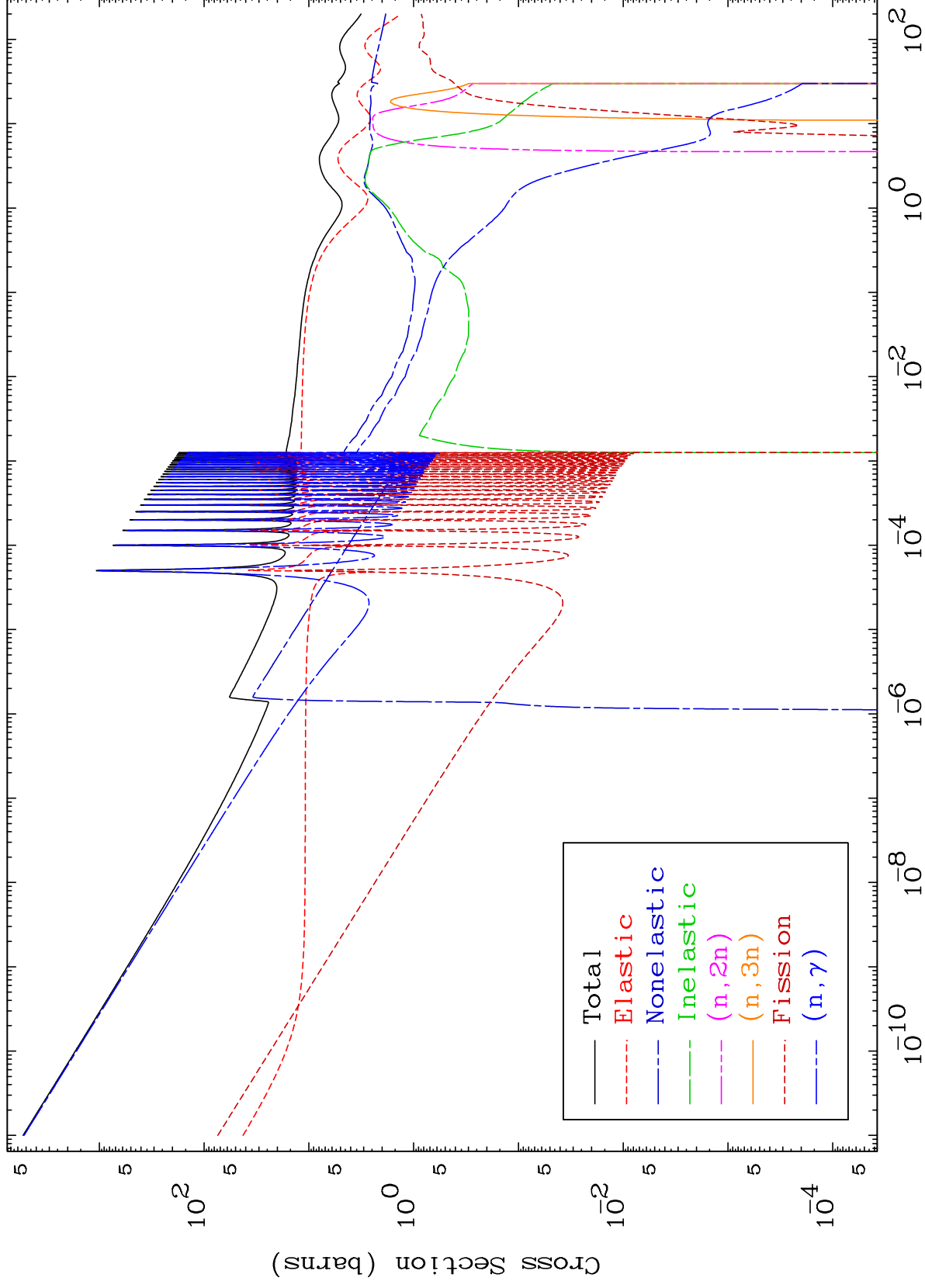
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Web:redcullen1.net/HOMEPAGE.NEW

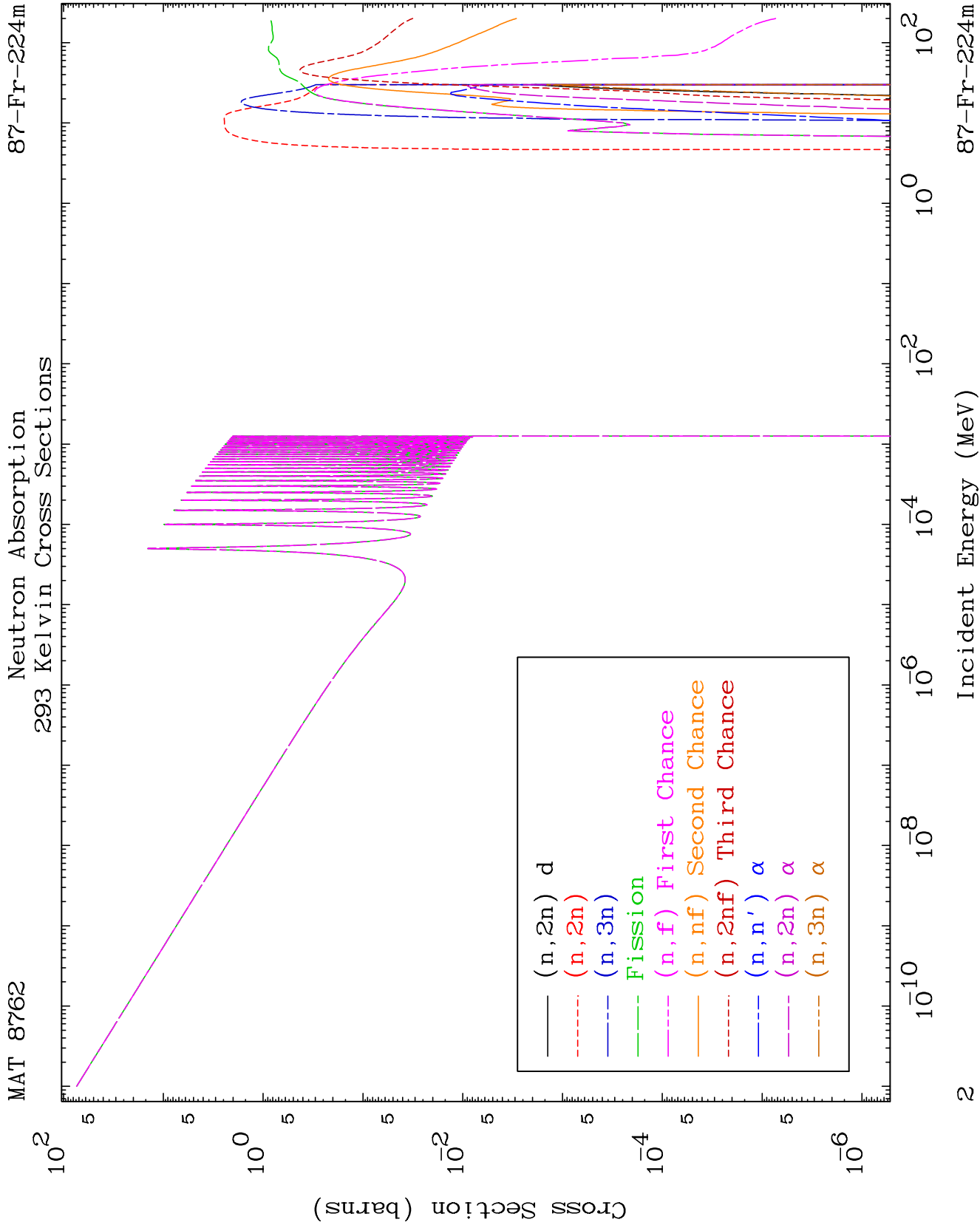
Press Mouse Button to Start

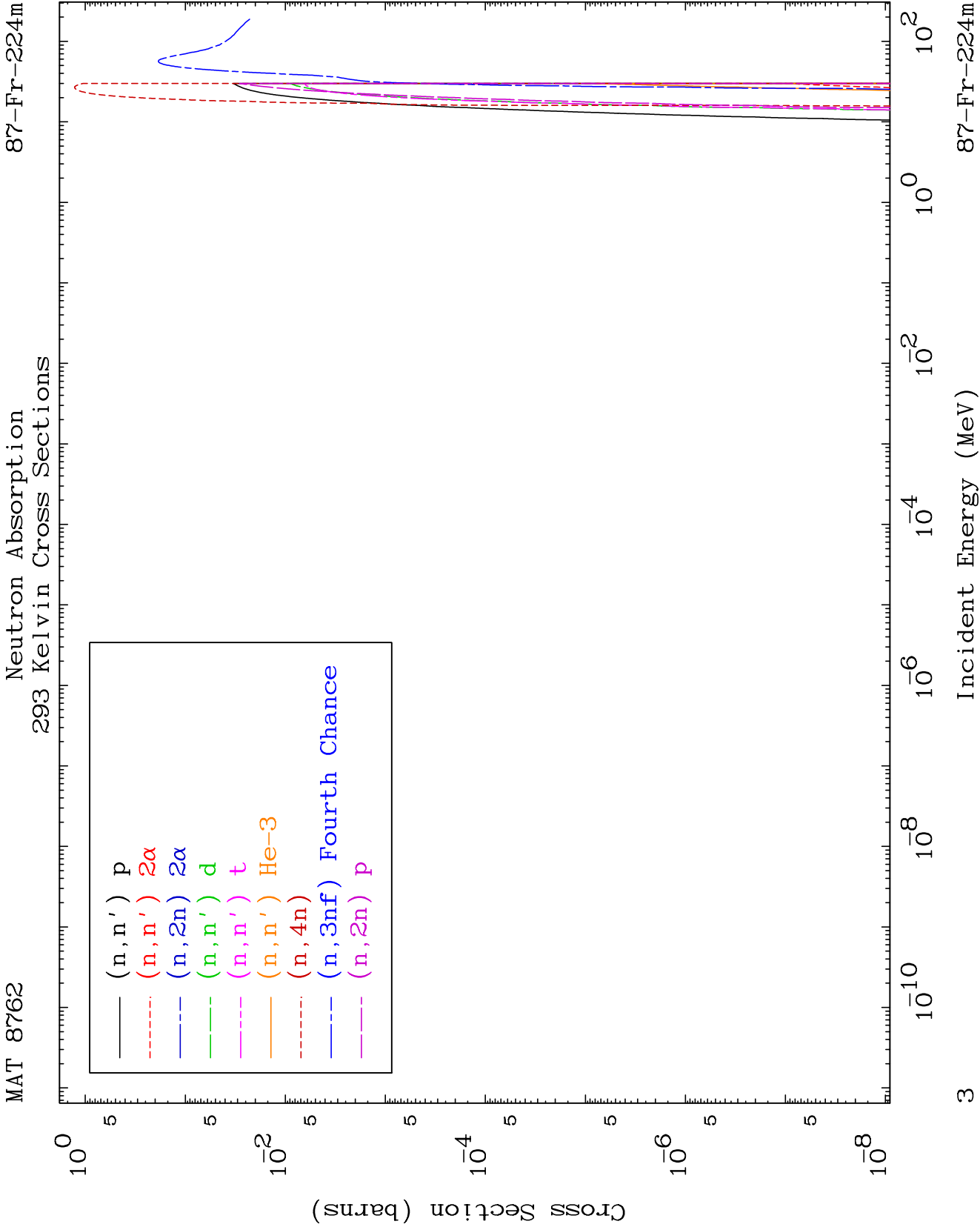
MAT 8762

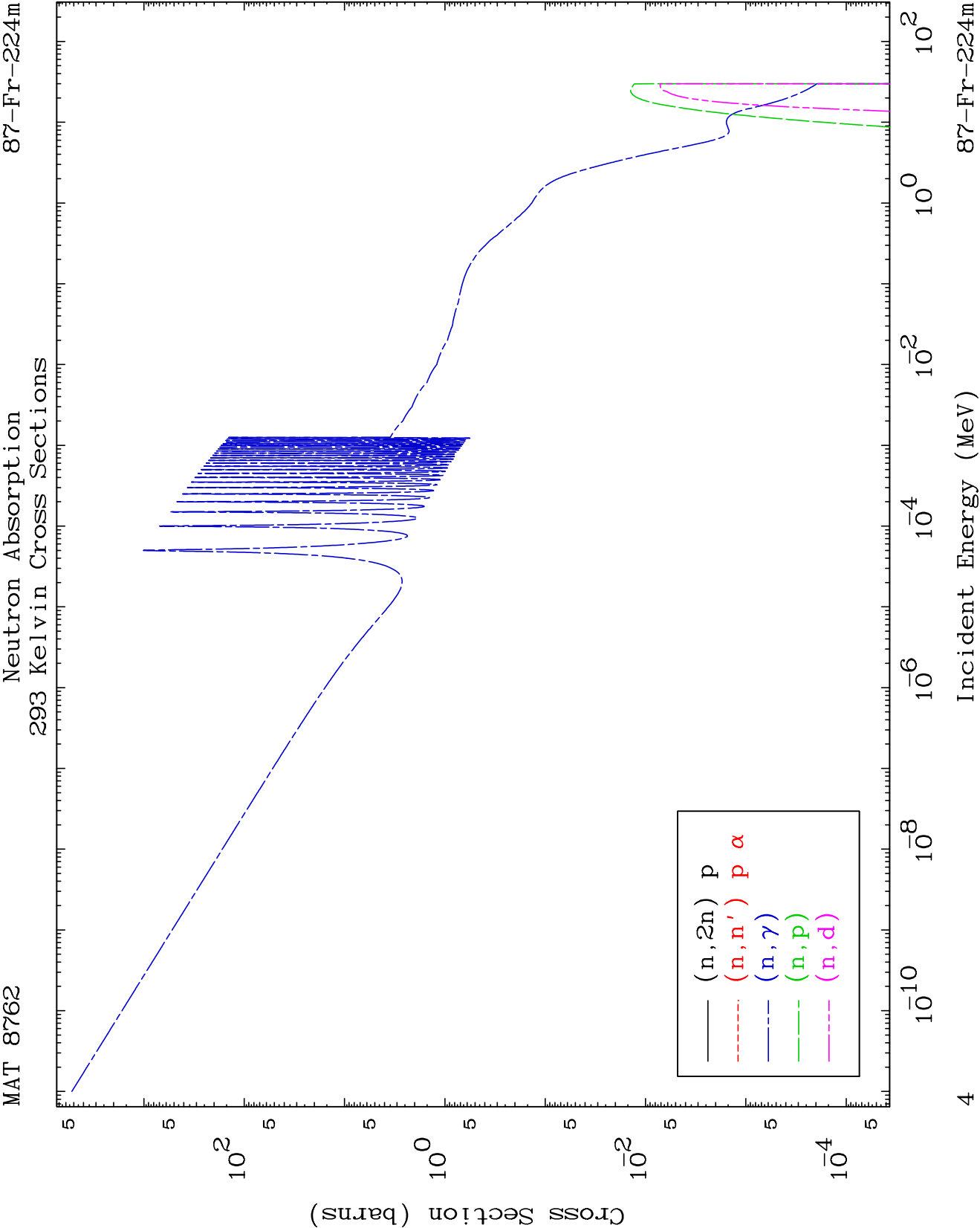
Neutron Major
293 Kelvin Cross Sections

87-Fr-224m





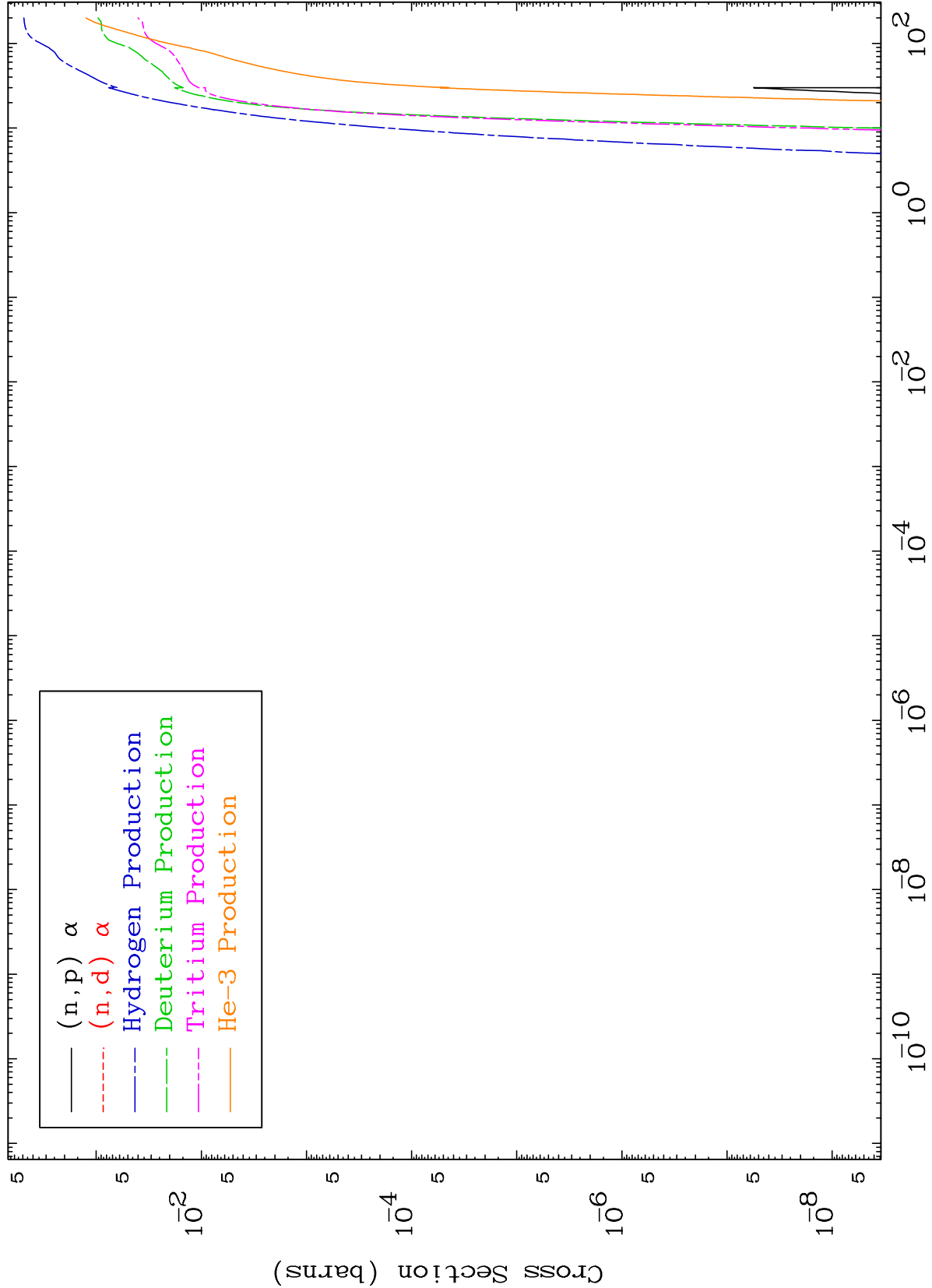




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Neutron Absorption
293 Kelvin Cross Sections

87-Fr-224m



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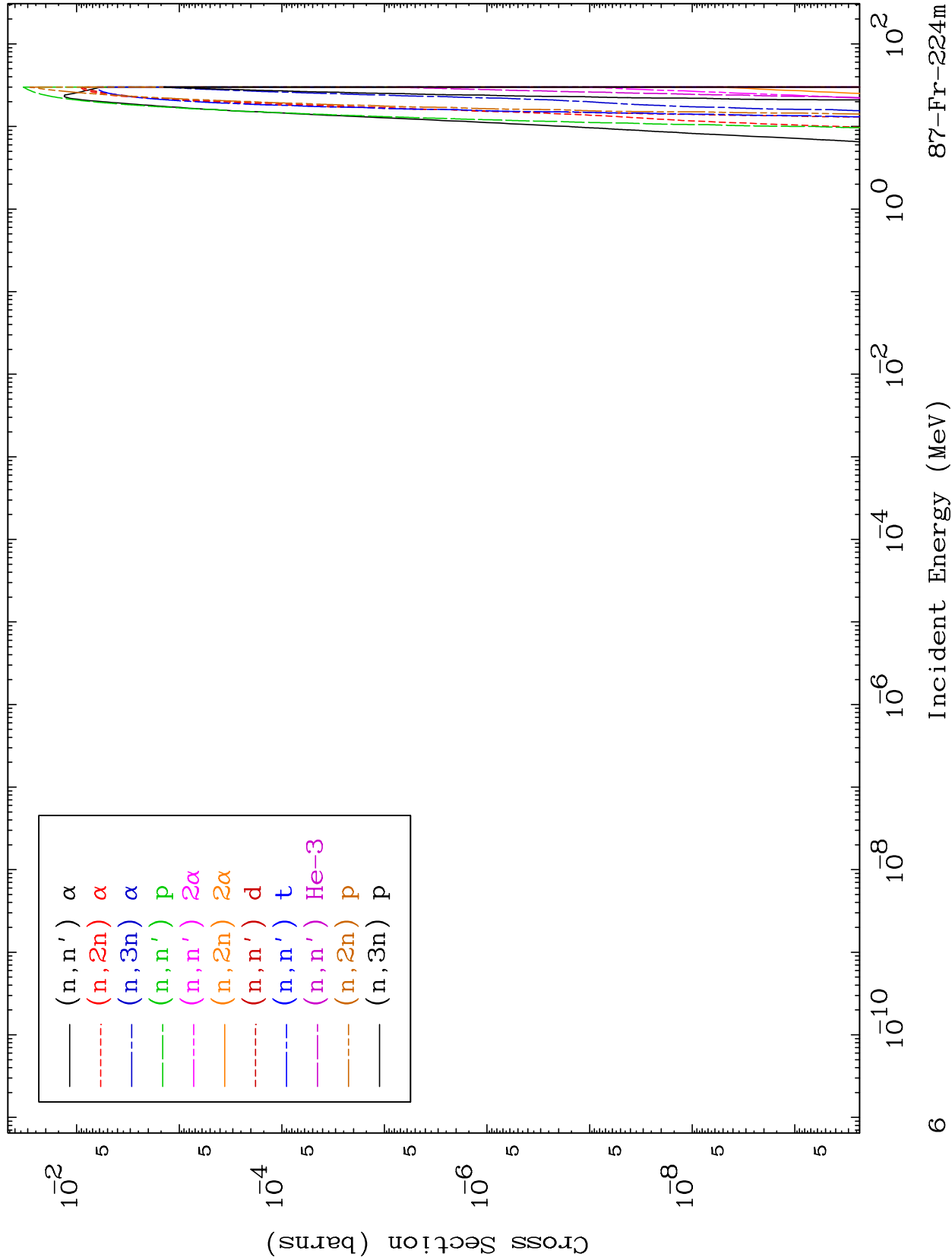
Incident Energy (MeV)

87-Fr-224m

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Charged Particle
293 Kelvin Cross Sections

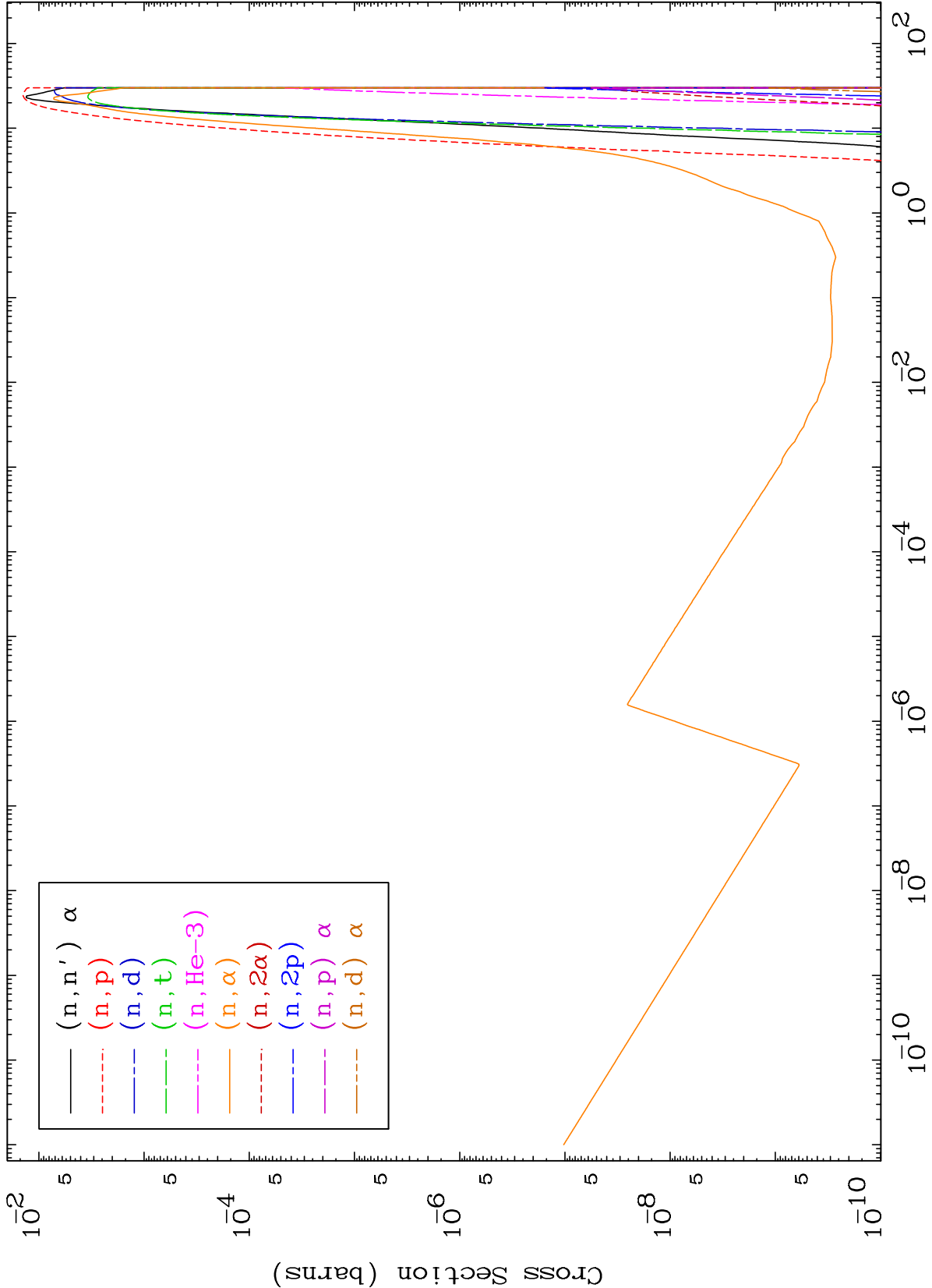
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Charged Particle
293 Kelvin Cross Sections

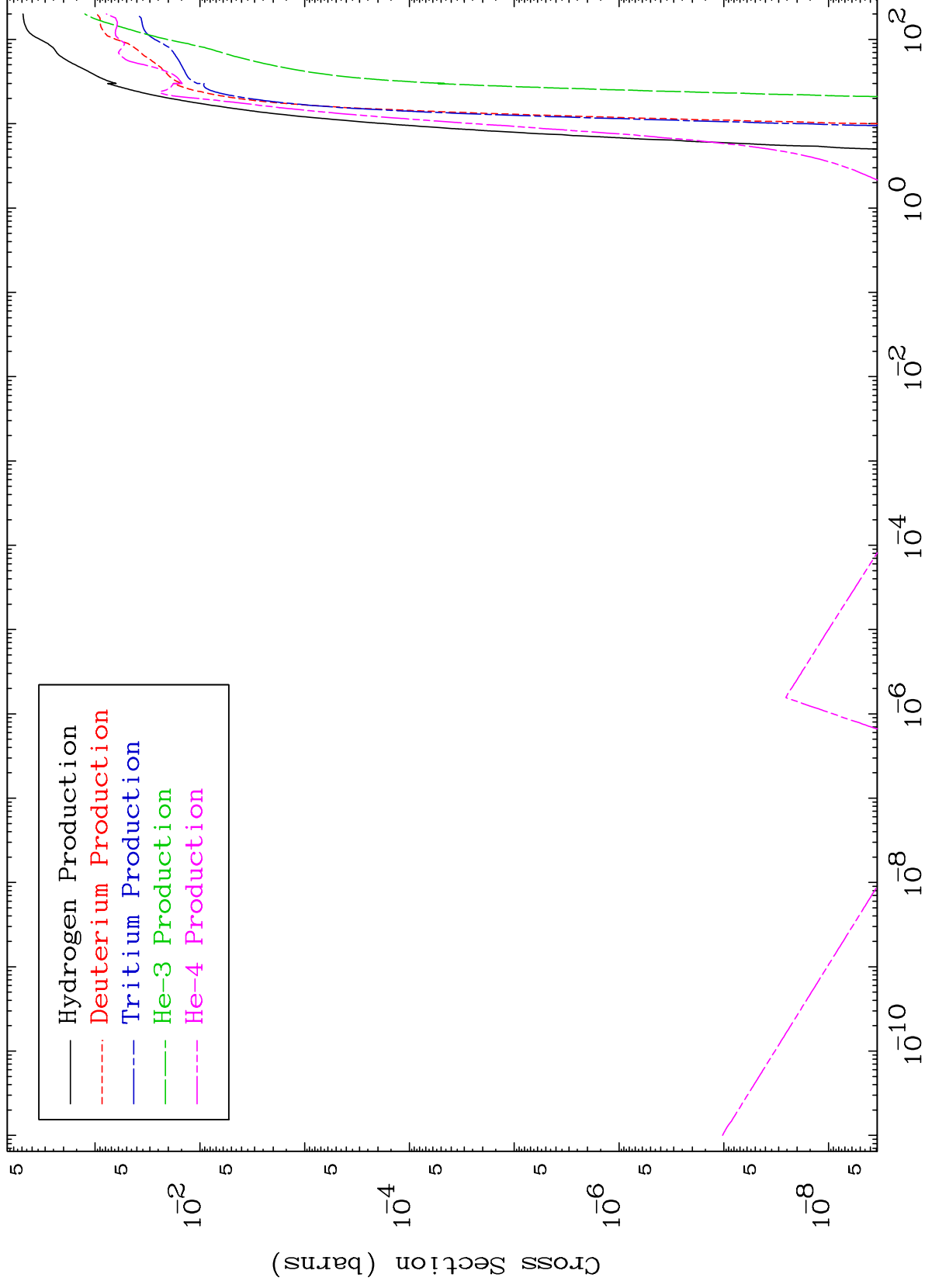
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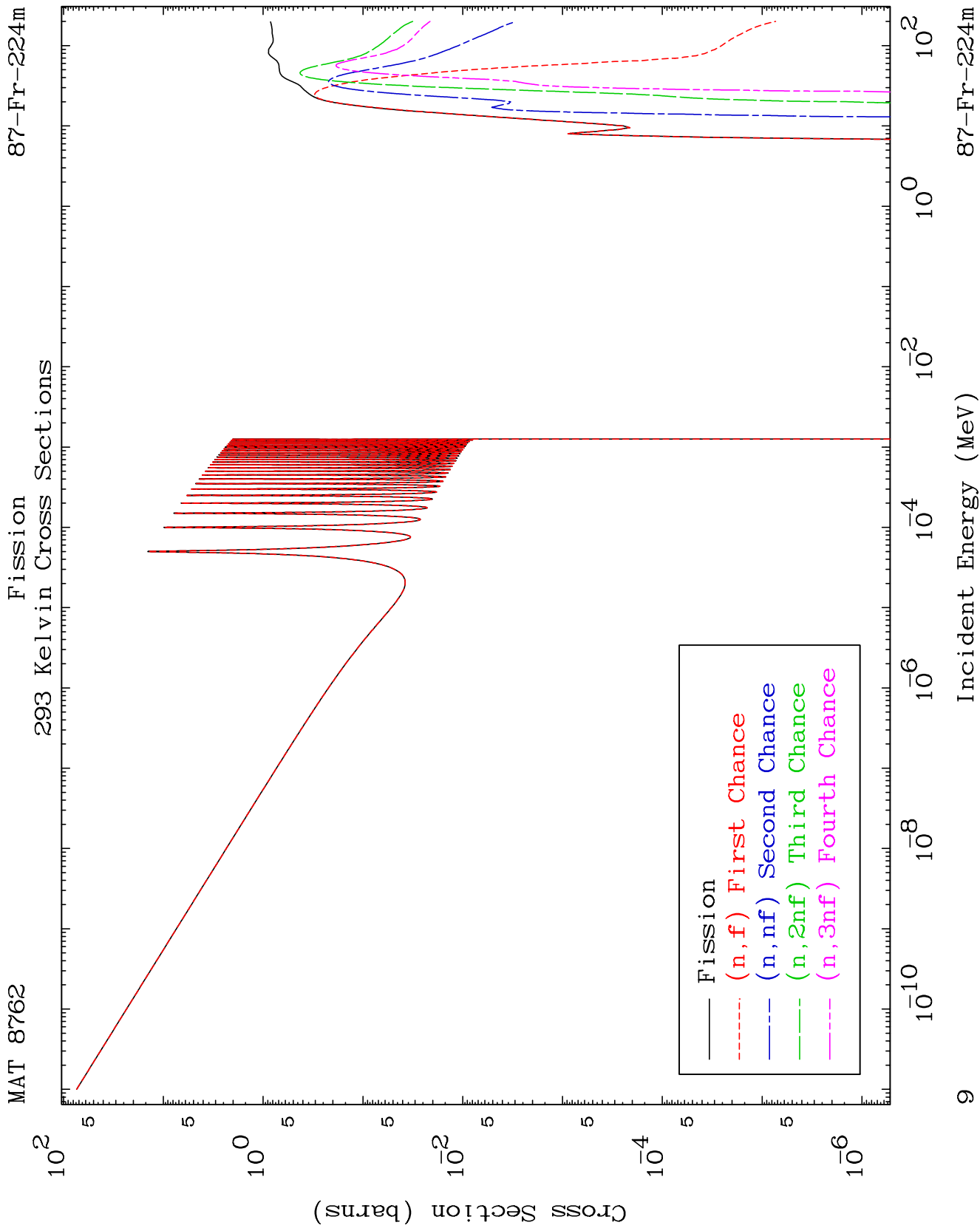


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Particle Production
293 Kelvin Cross Sections

87-Fr-224m

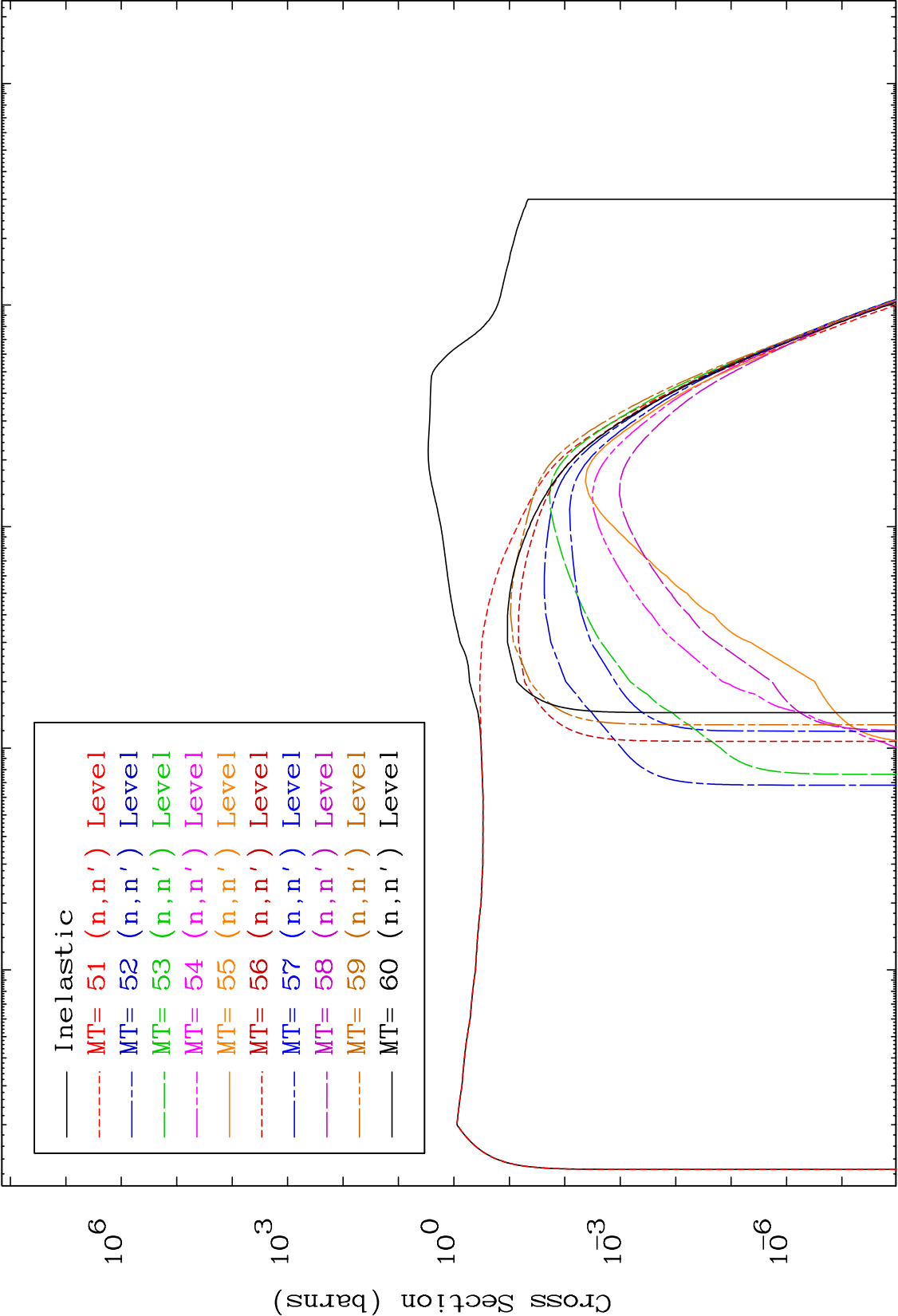




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(n,n') Levels
293 Kelvin Cross Sections

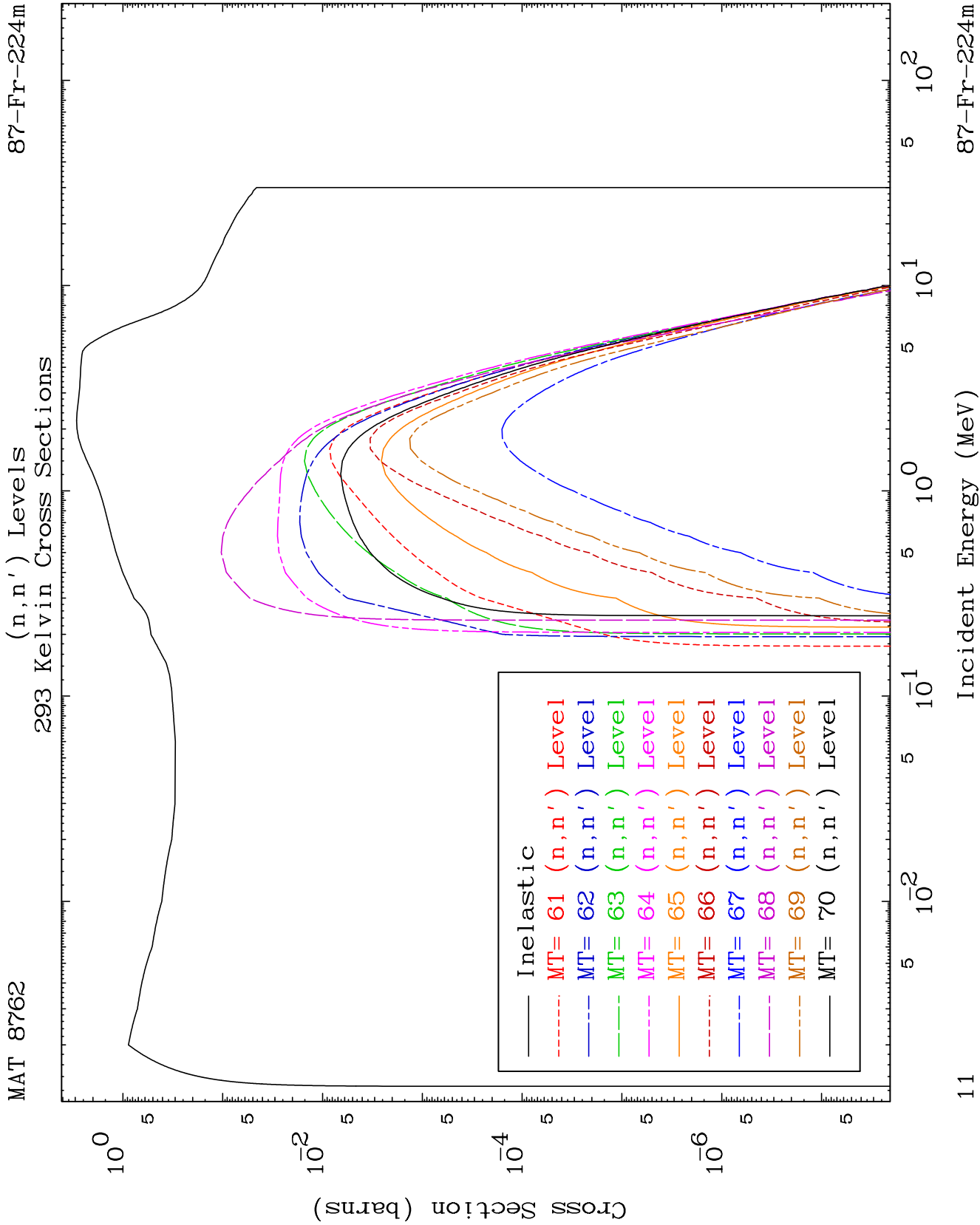
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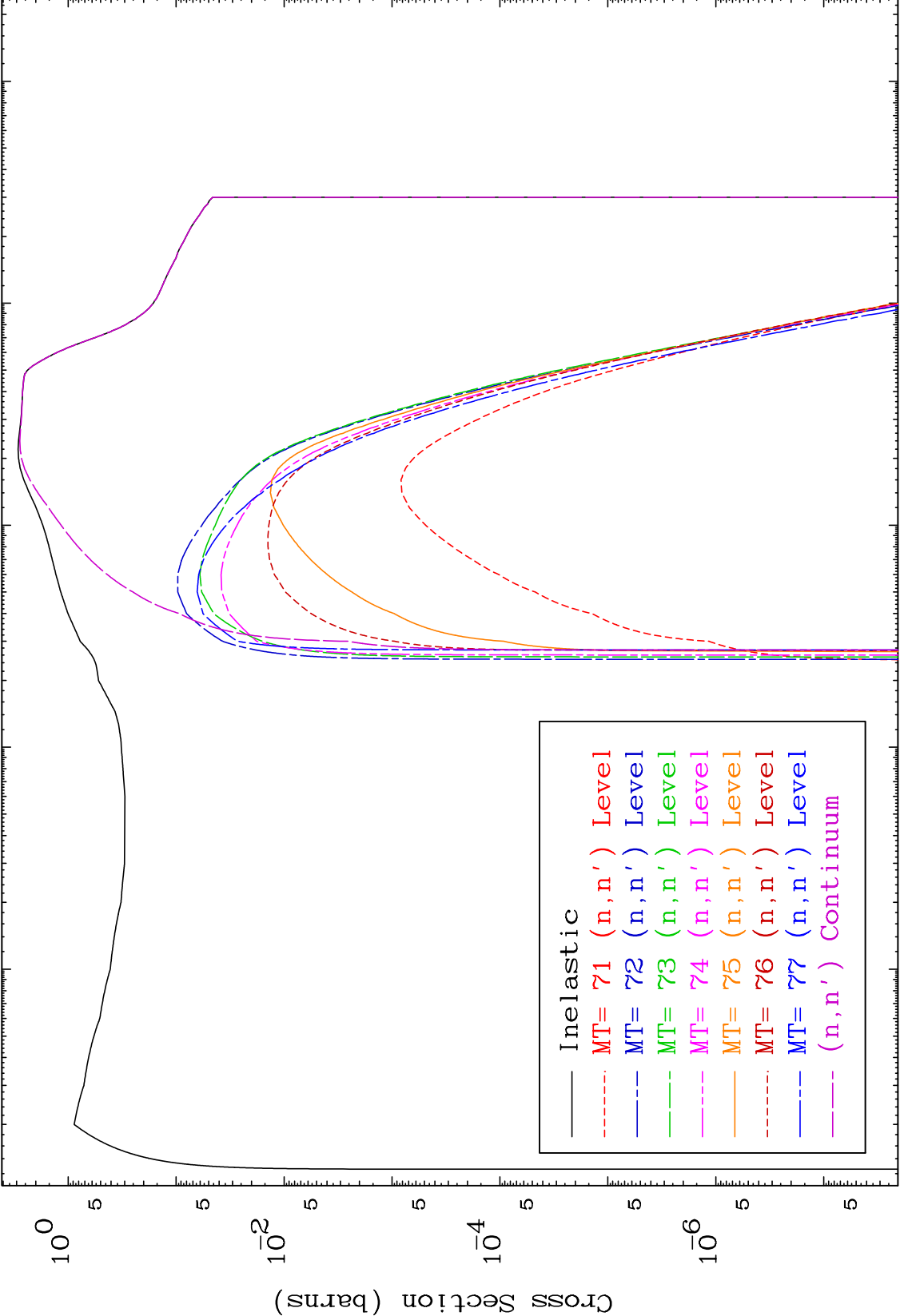


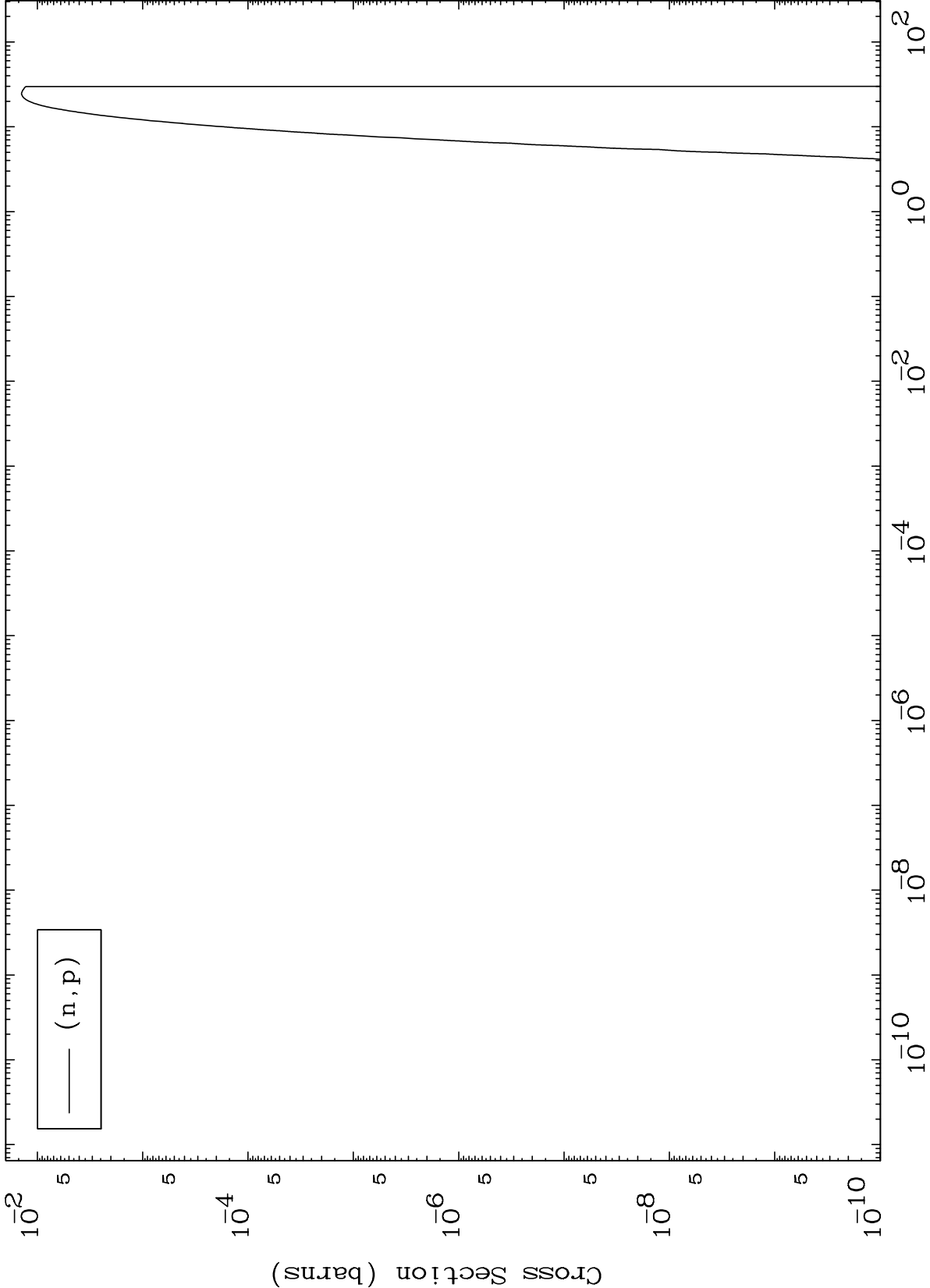
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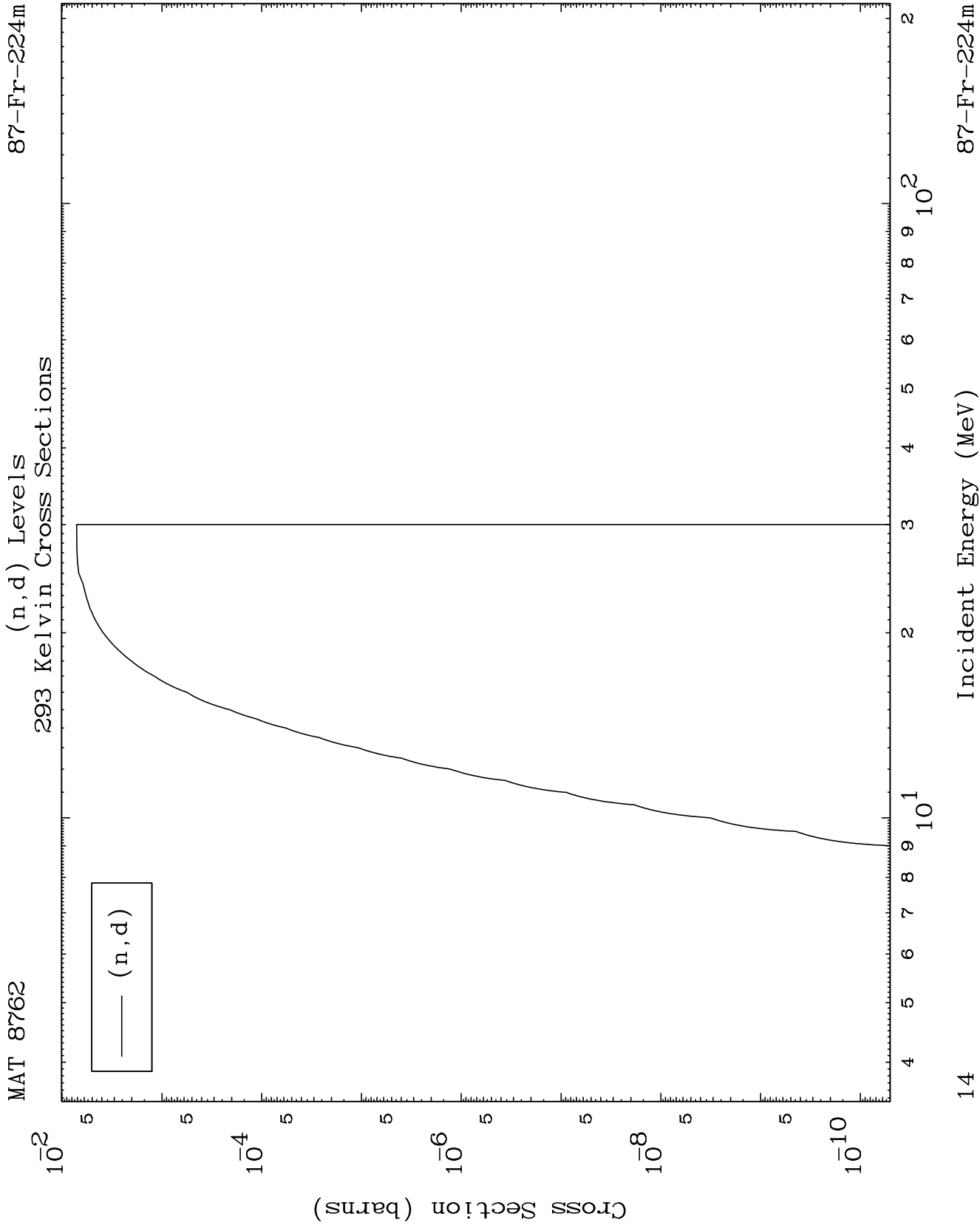
Incident Energy (MeV)

87-Fr-224m





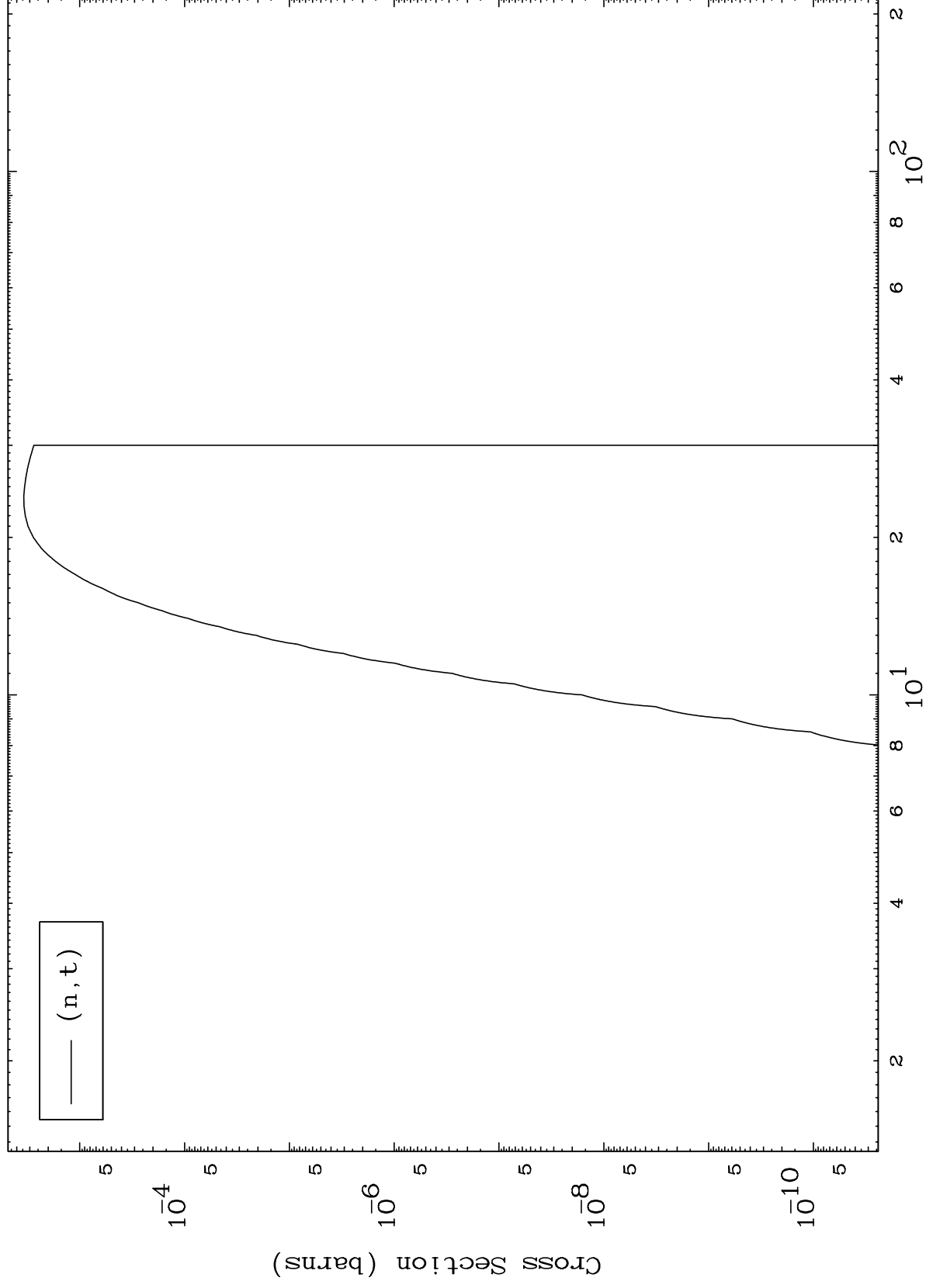




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(n,t) Levels
293 Kelvin Cross Sections

87-Fr-224m



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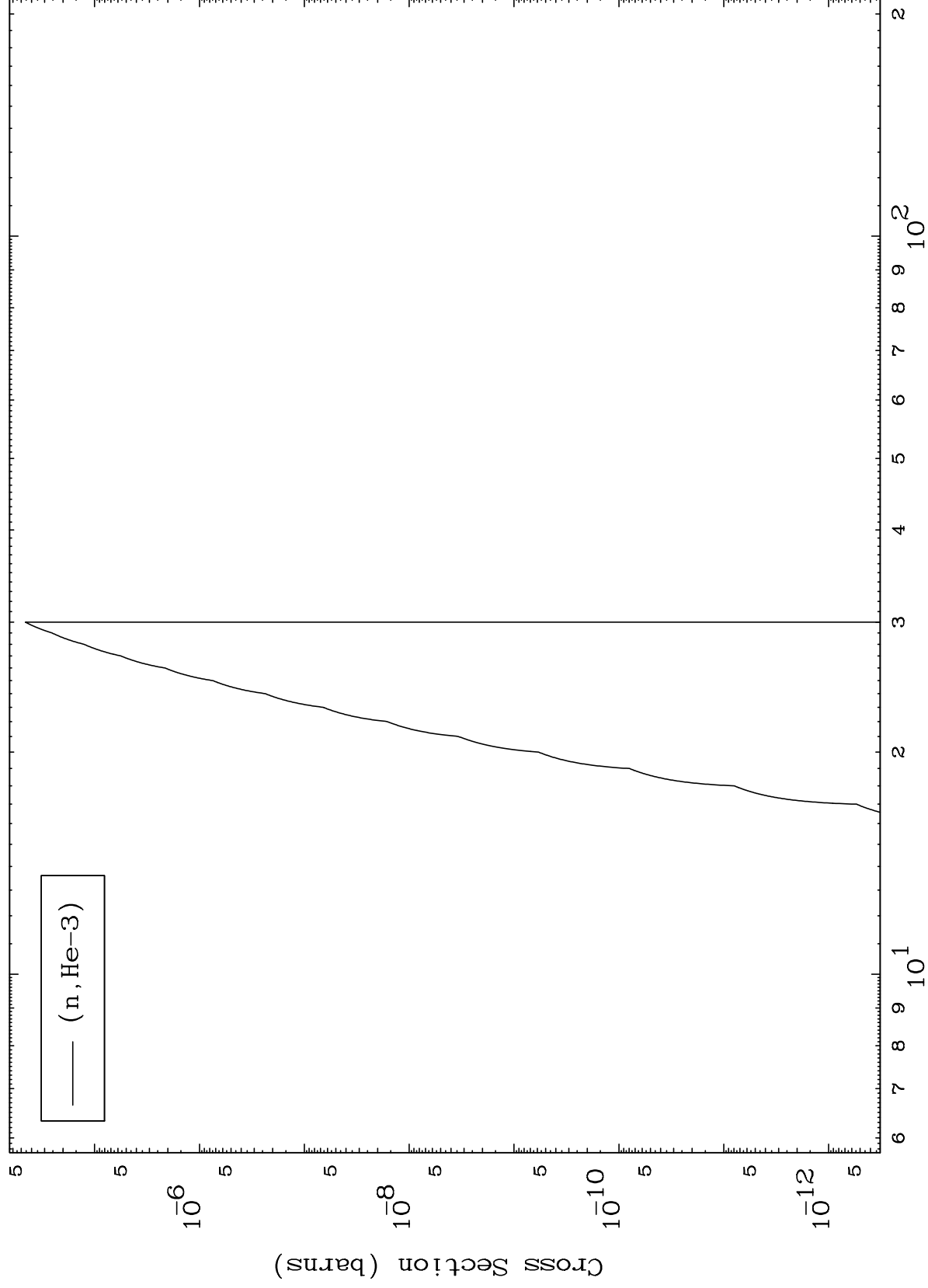
Incident Energy (MeV)

87-Fr-224m

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87-Fr-224m

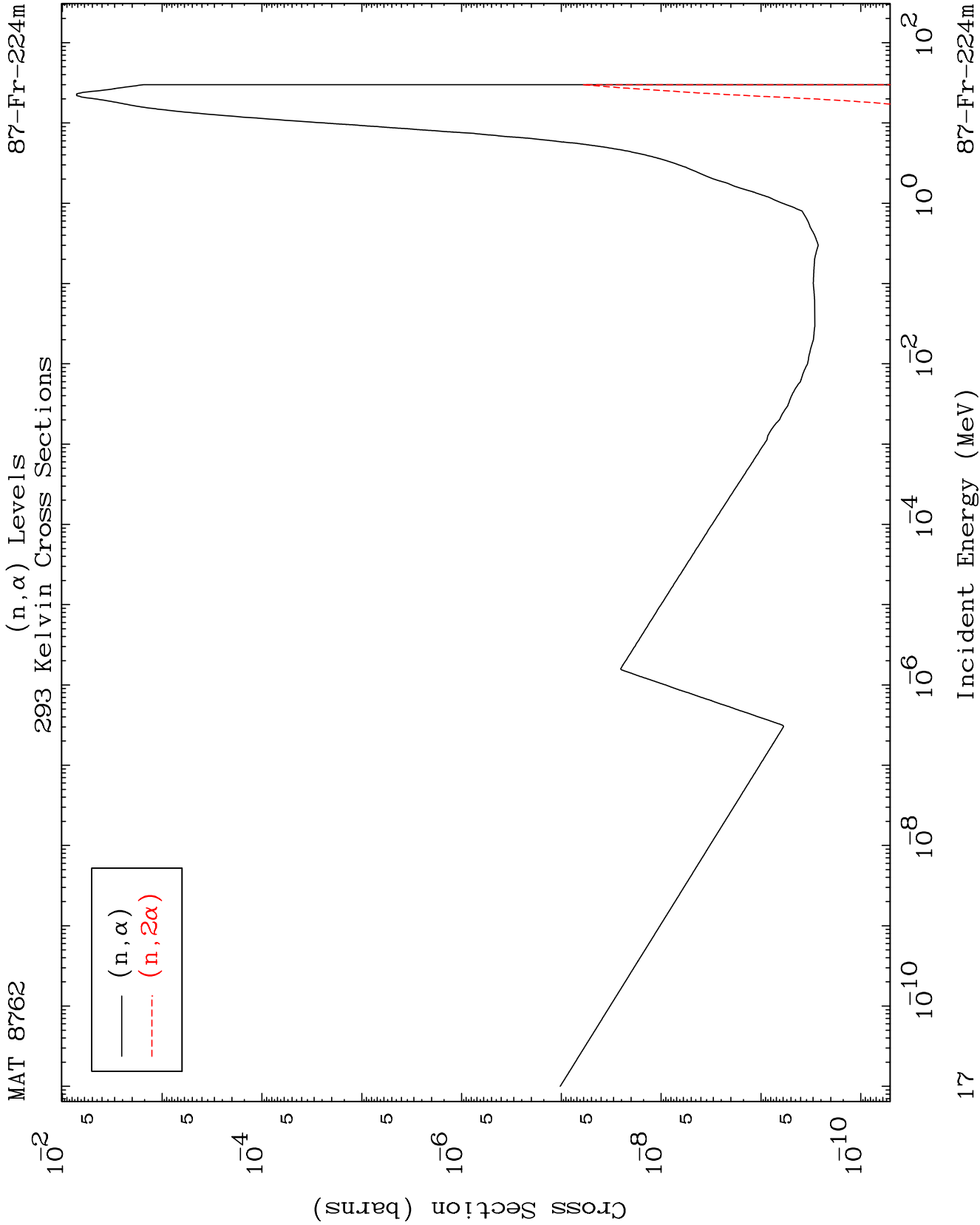
(n,He3) Levels
293 Kelvin Cross Sections

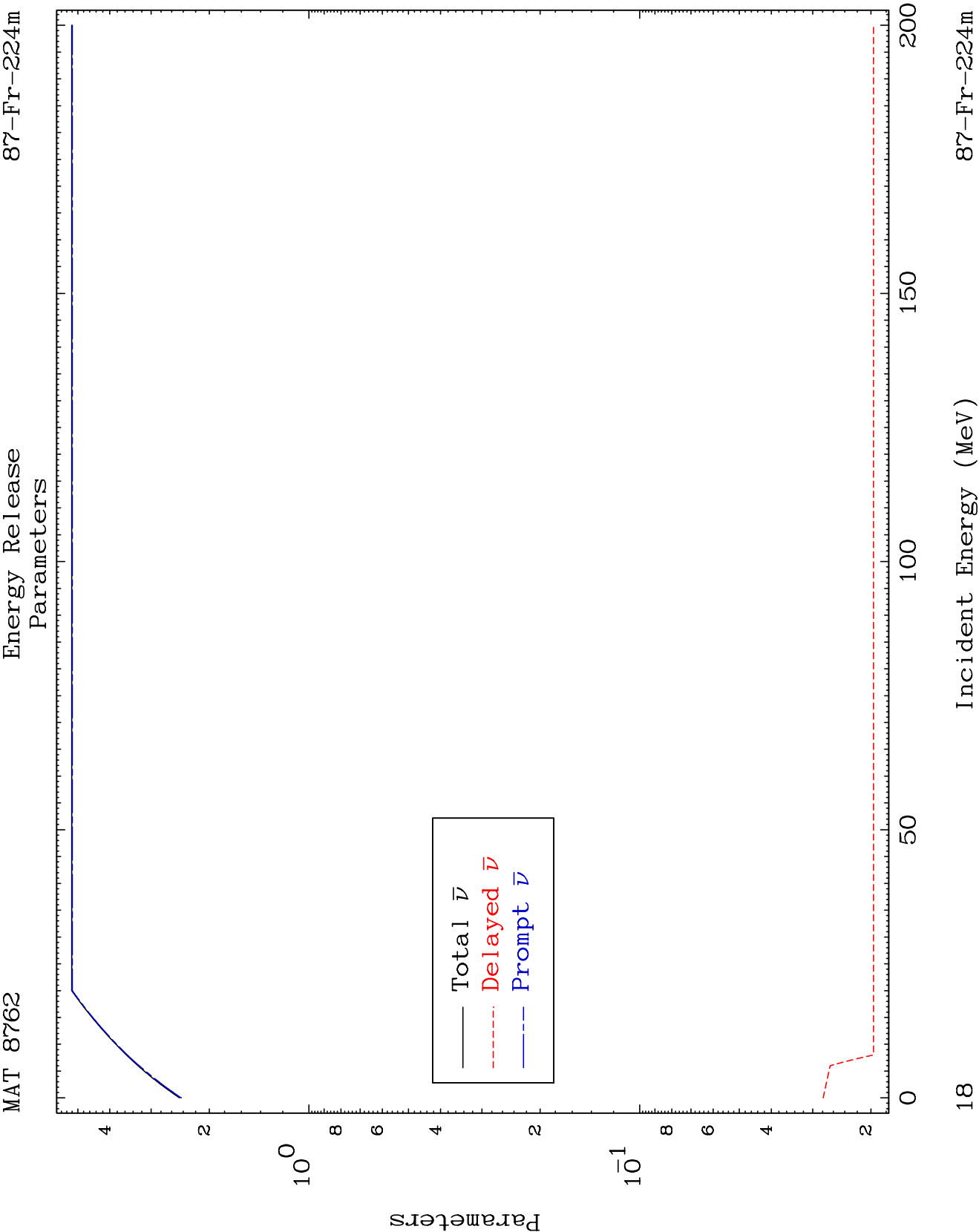


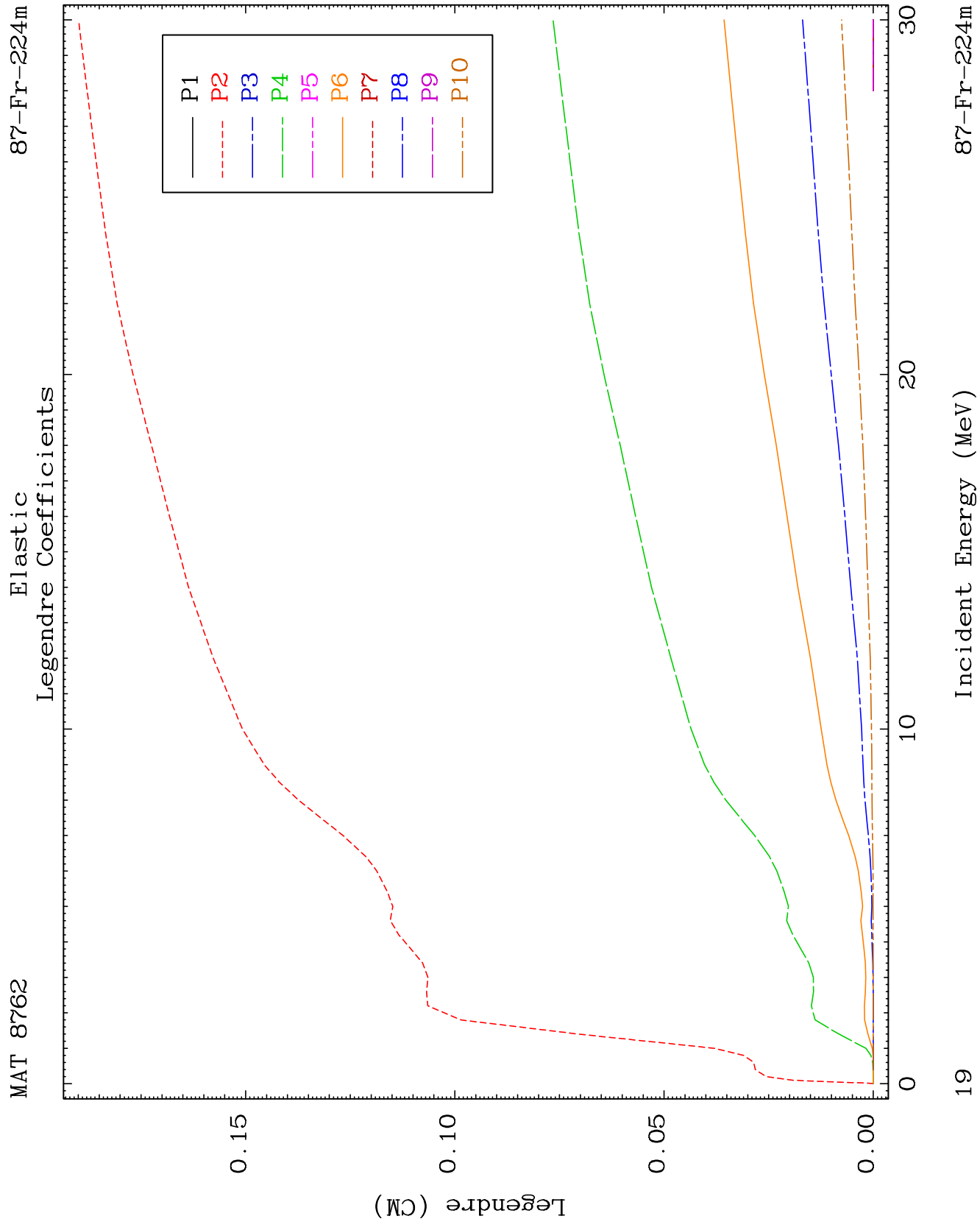
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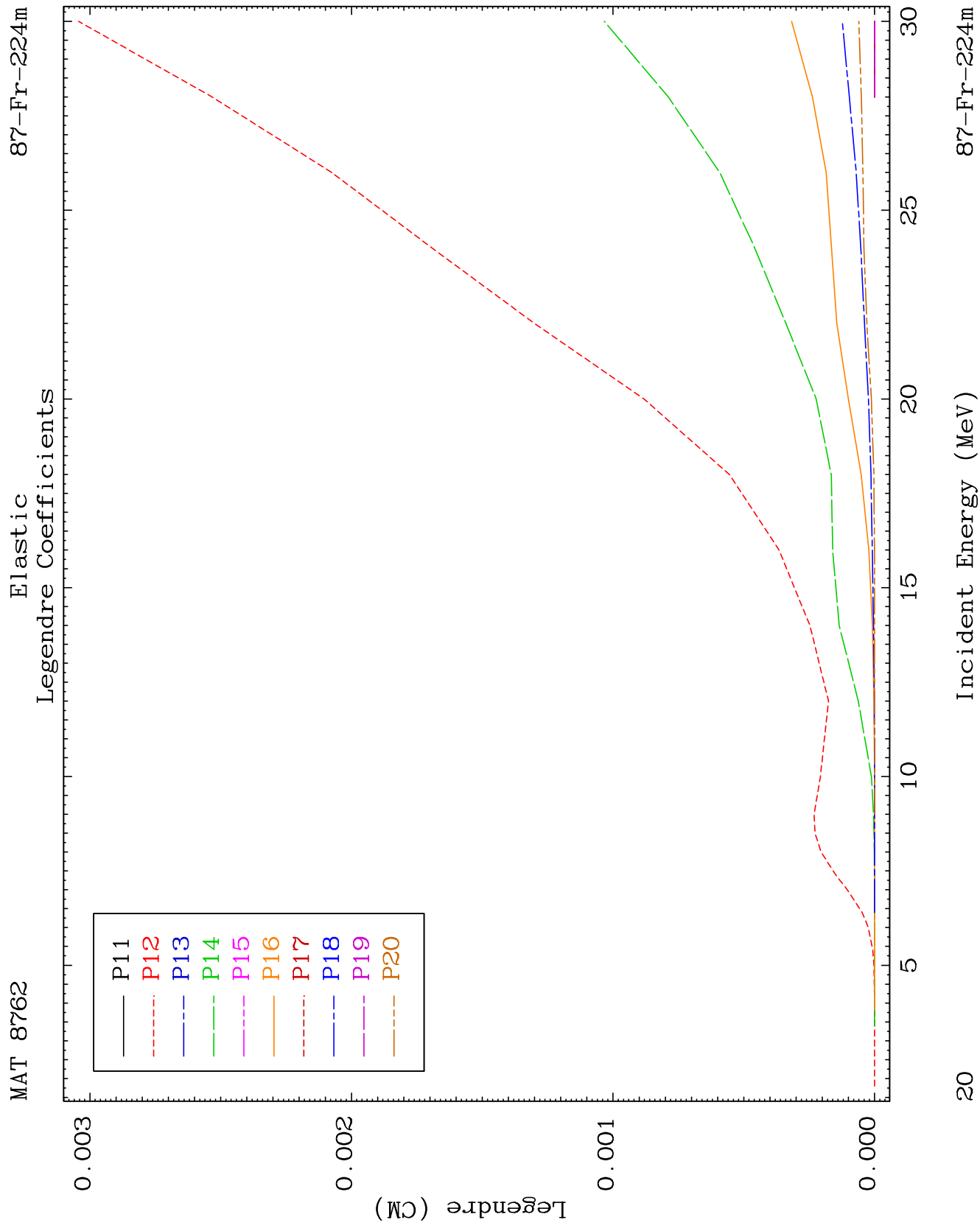
Incident Energy (MeV)

87-Fr-224m





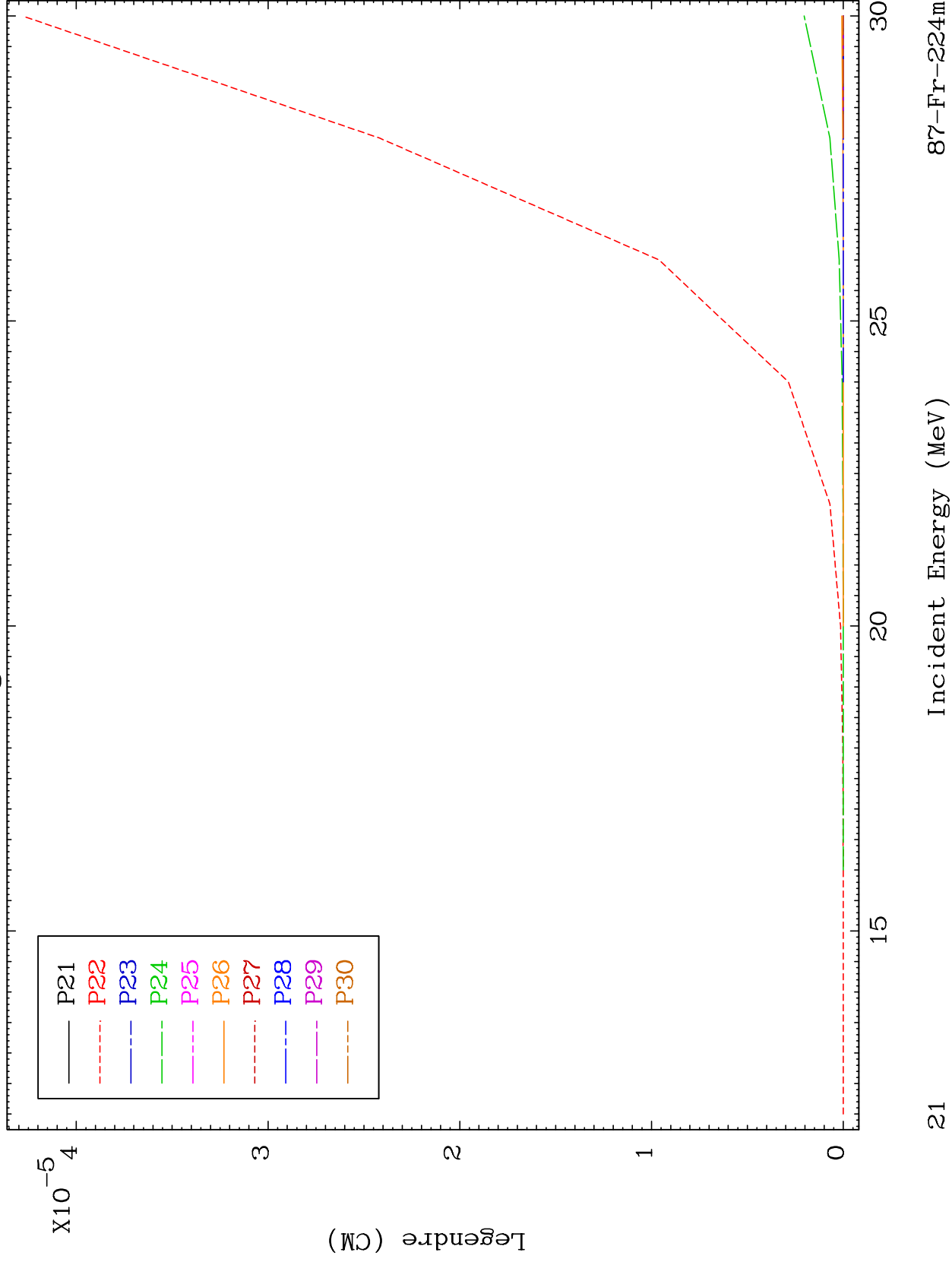




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Elastic Legendre Coefficients

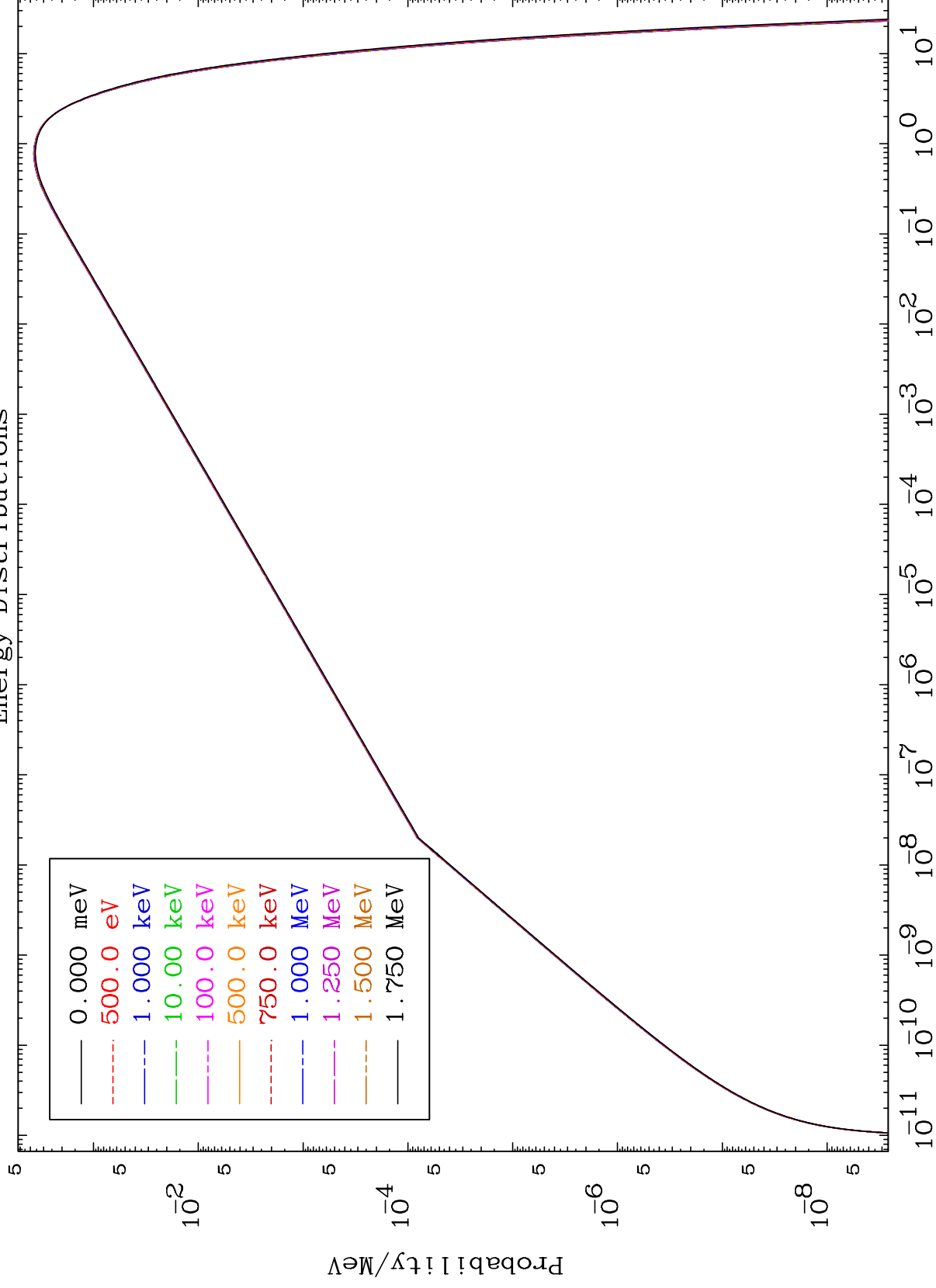
87-Fr-224m



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Fission
Energy Distributions

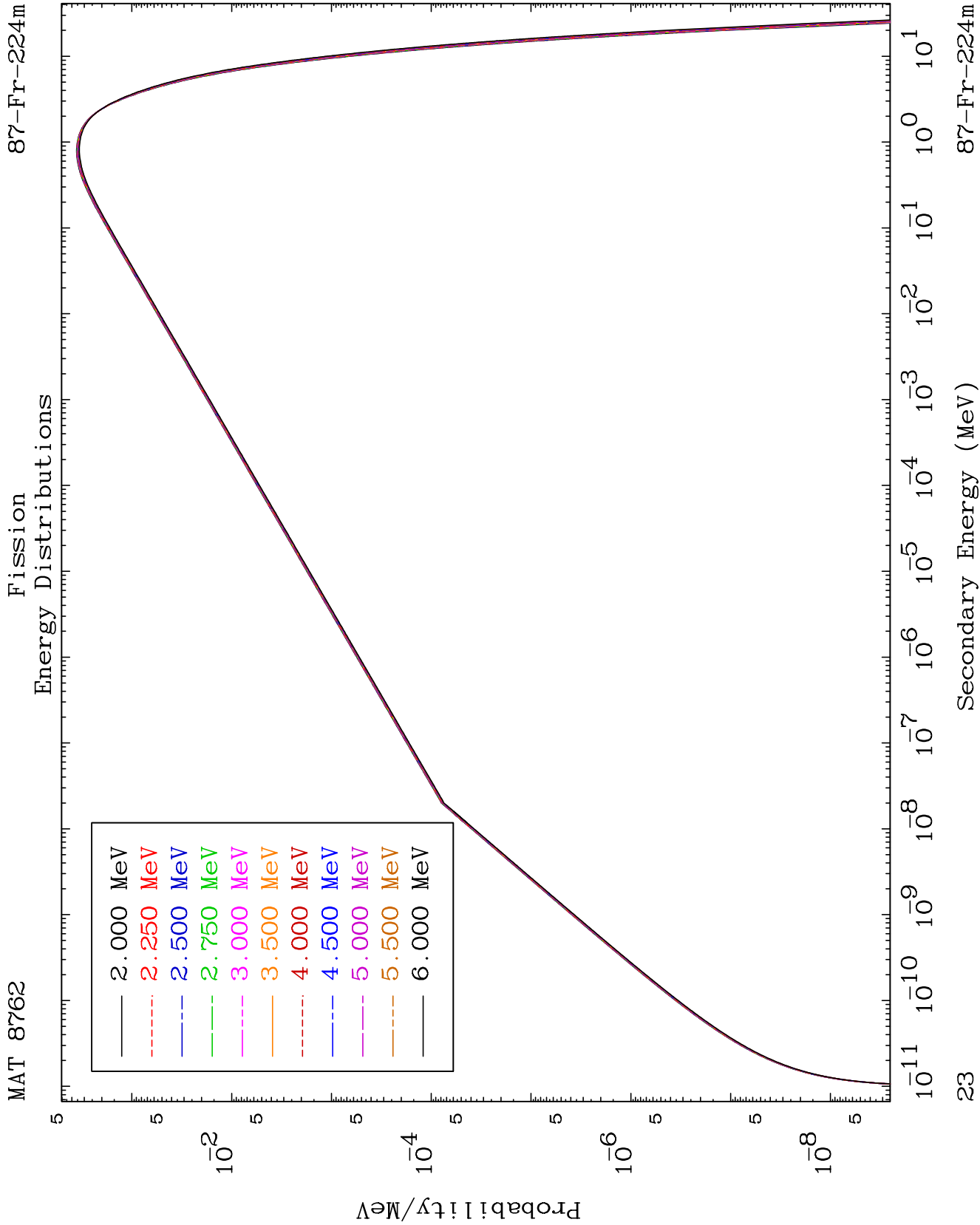
87-Fr-224m



Secondary Energy (MeV)

87-Fr-224m

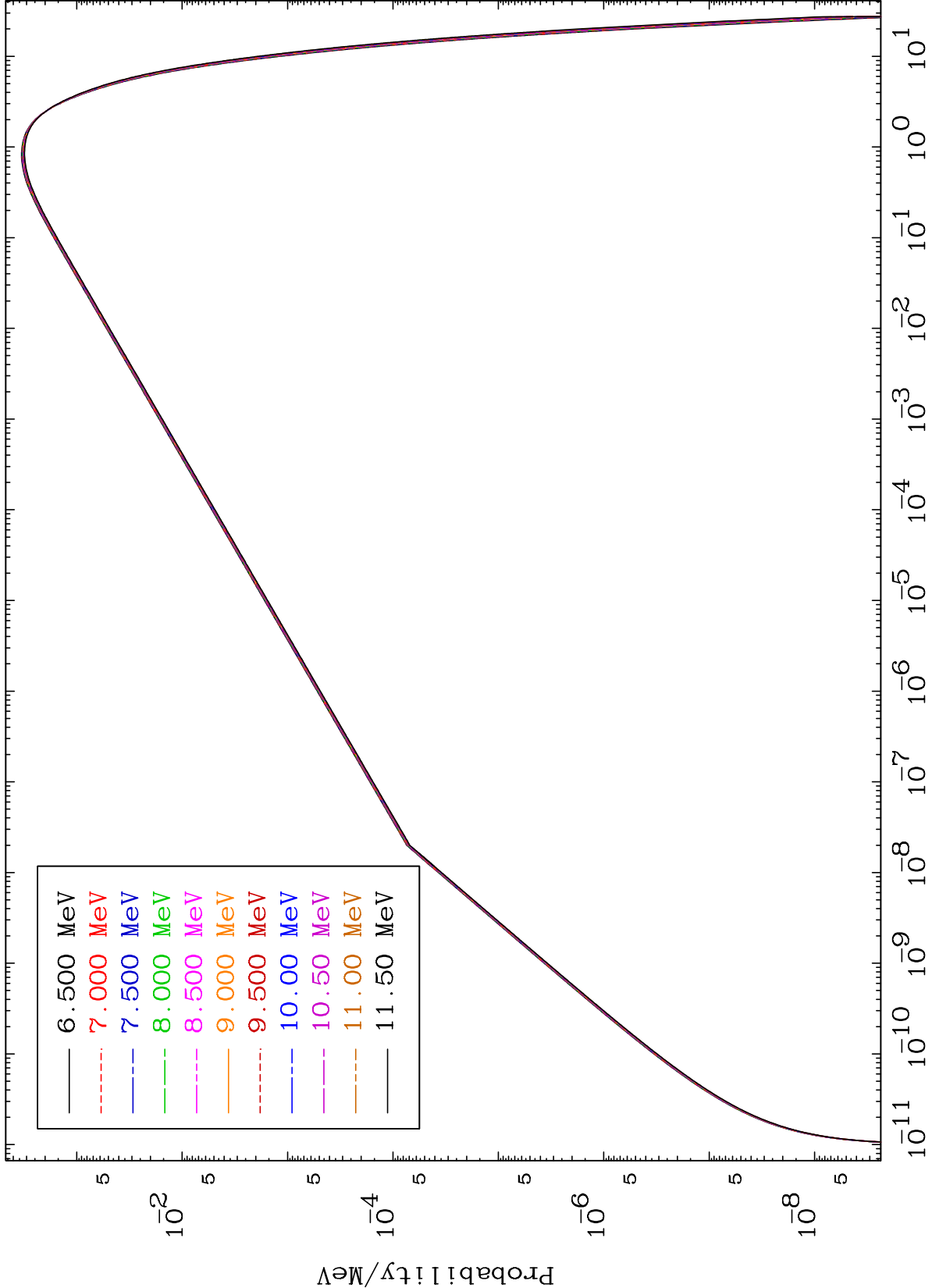
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Fission
Energy Distributions

87-Fr-224m



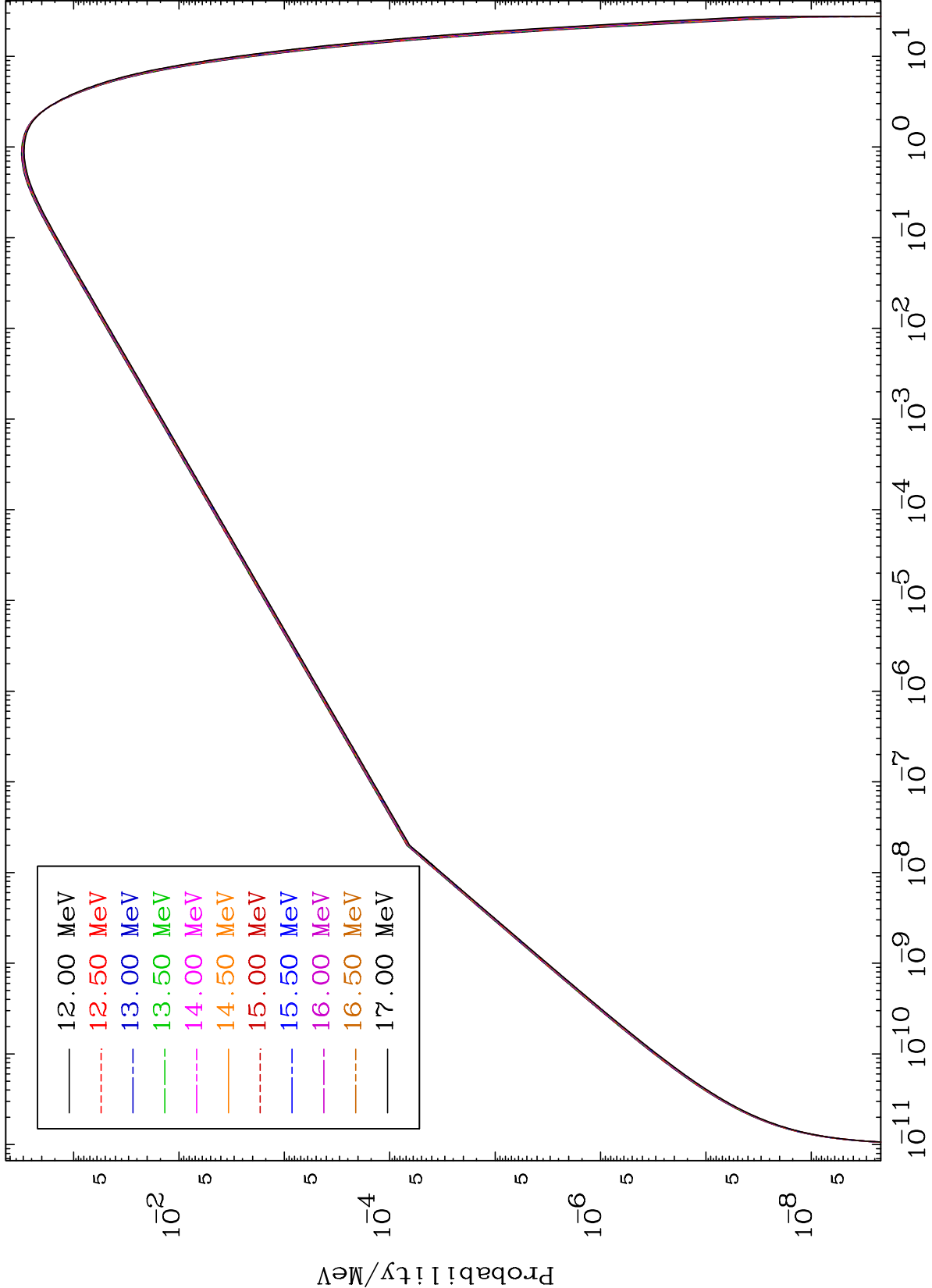
87-Fr-224m

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Fission
Energy Distributions

87-Fr-224m



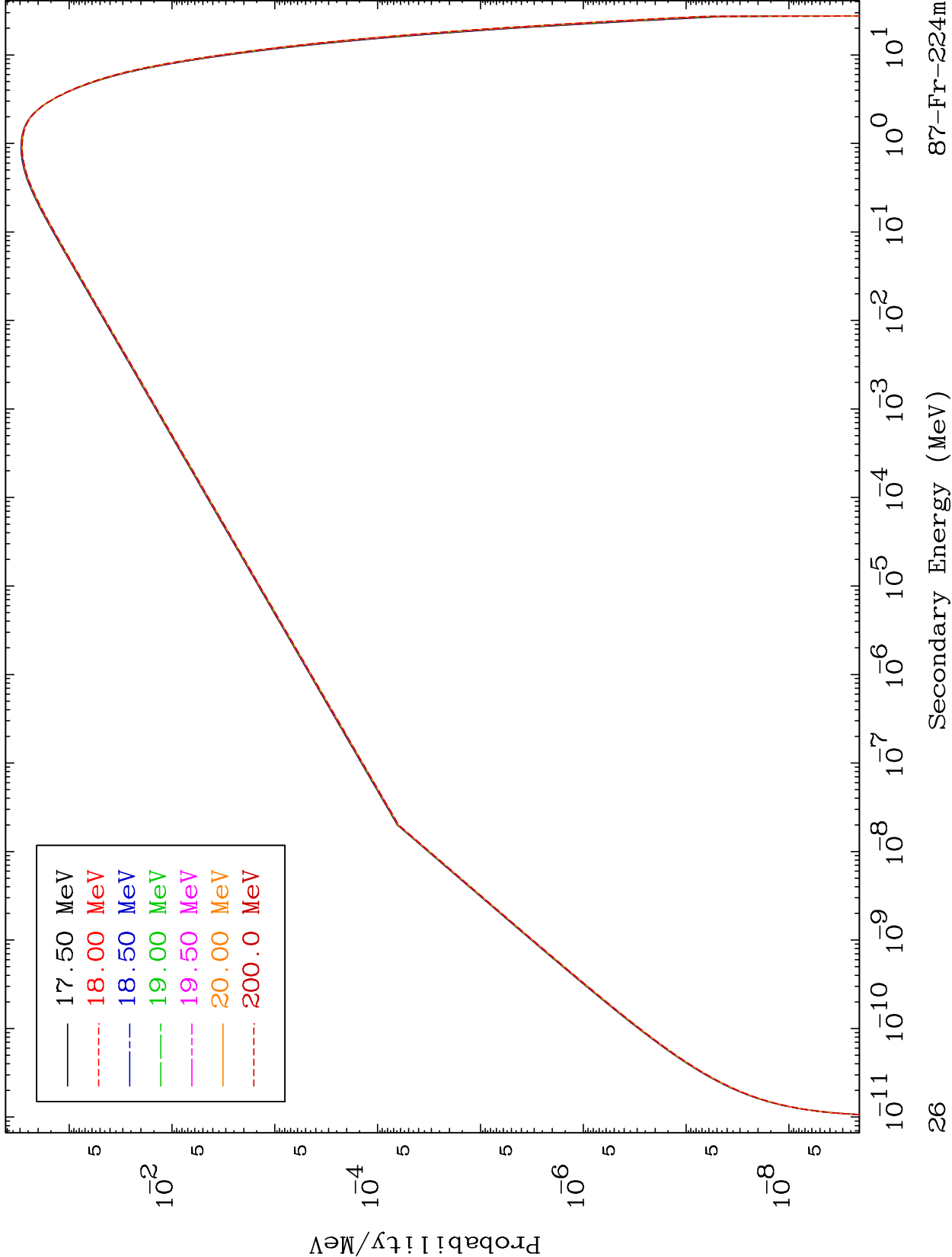
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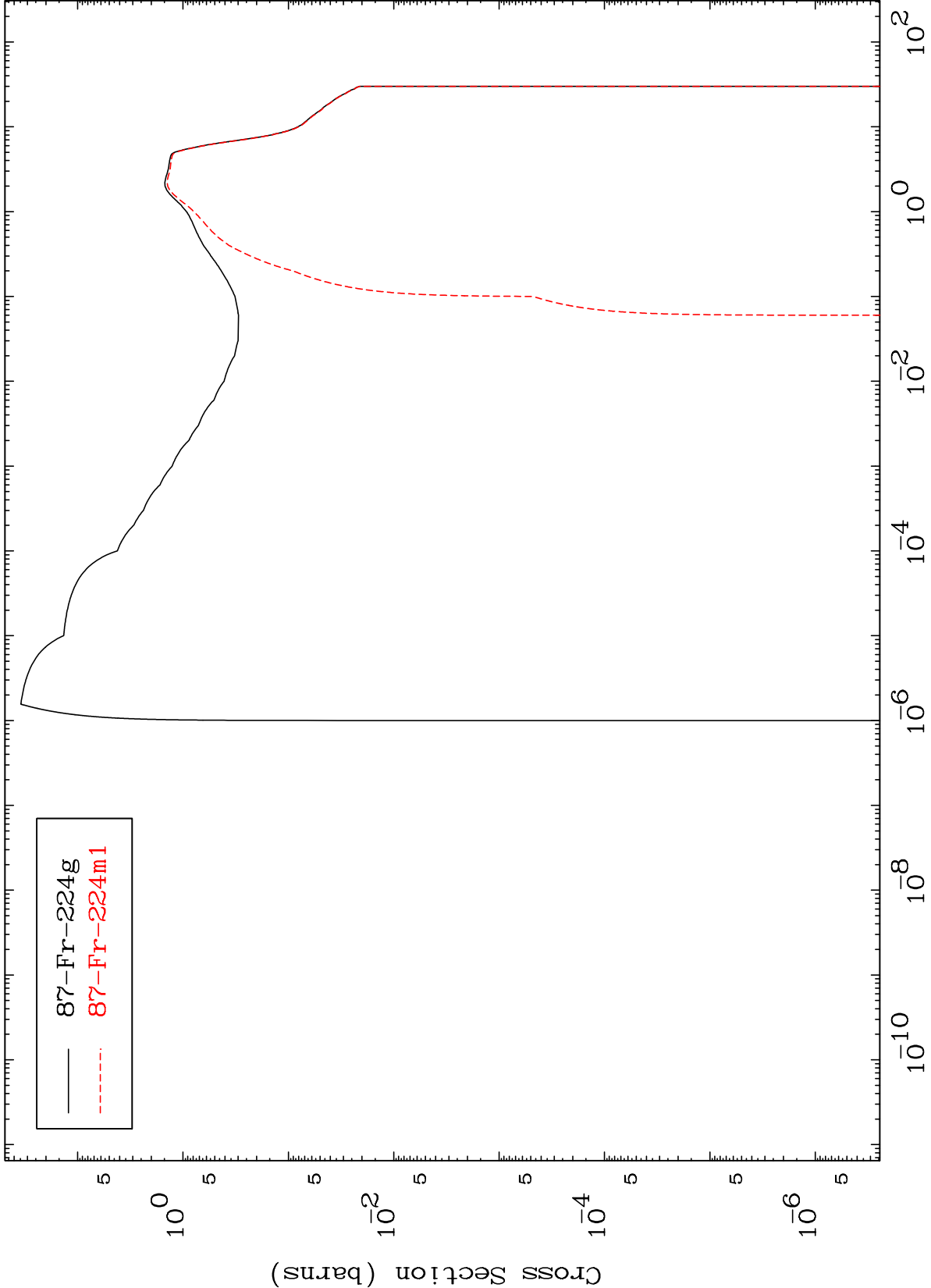
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Fission
Energy Distributions

87-Fr-224m



Inelastic
Radionuclide Production Cross Section

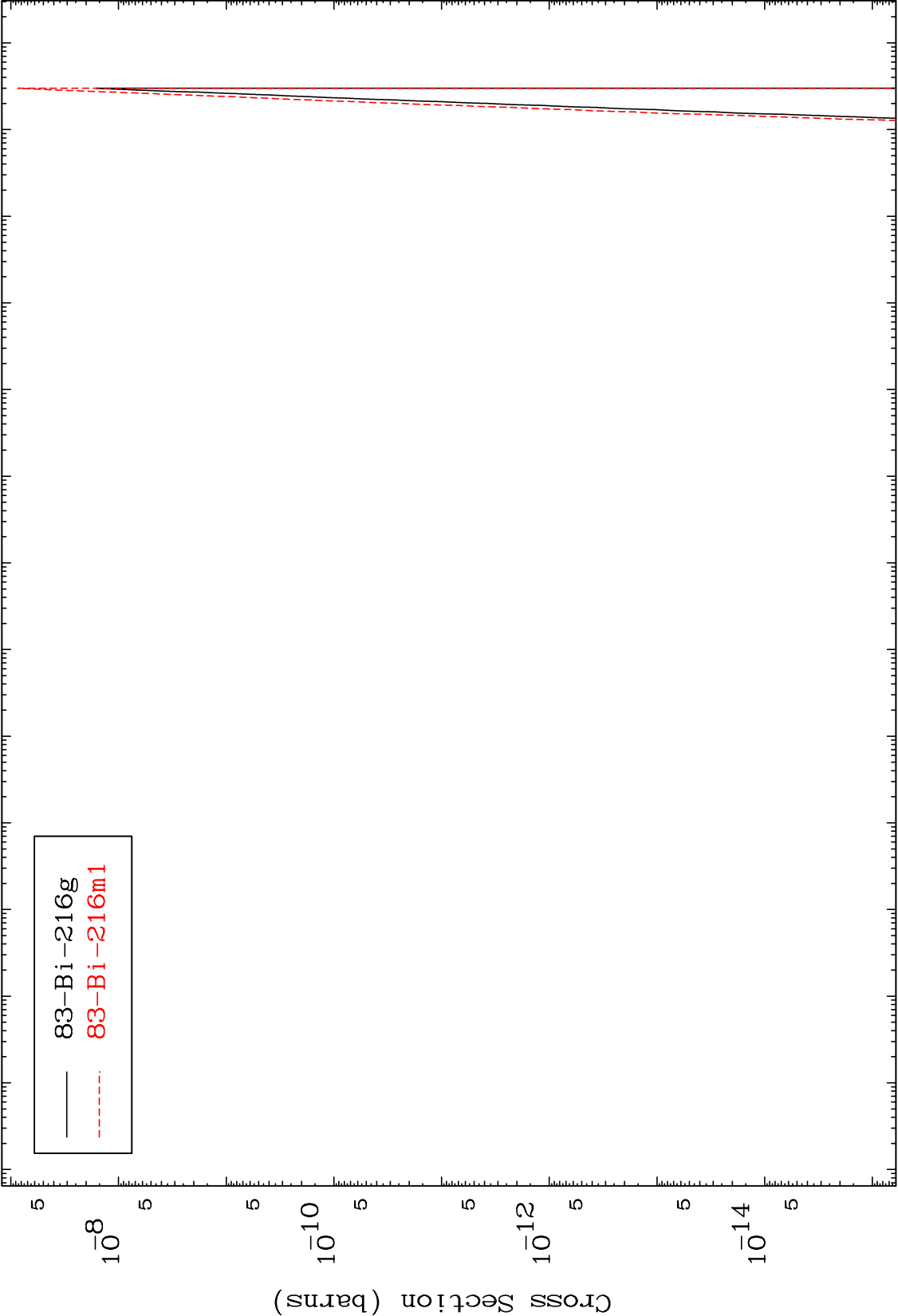
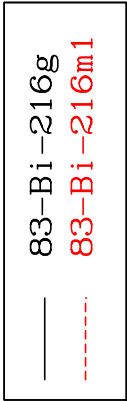


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(n,n') 2 α

87-Fr-224m

Radionuclide Production Cross Section



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(n,2n) 2 α

87-Fr-224m

Radionuclide Production Cross Section

