

Program EVALPLOT  
(Version 2021-1)

by

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(Present Contact Information)

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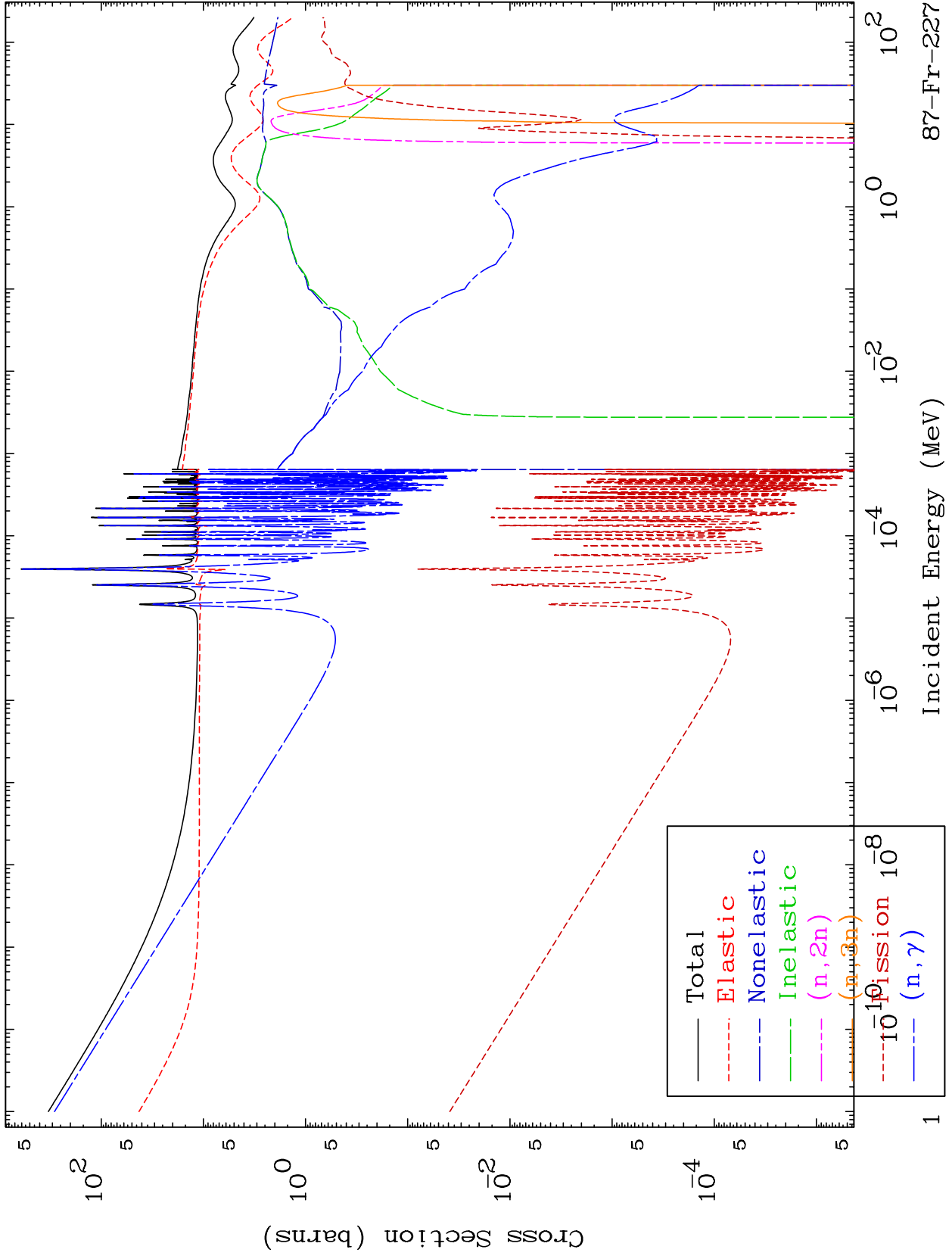
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Press Mouse Button to Start

MAT 8770

Neutron Major  
293 Kelvin Cross Sections

87-Fr-227

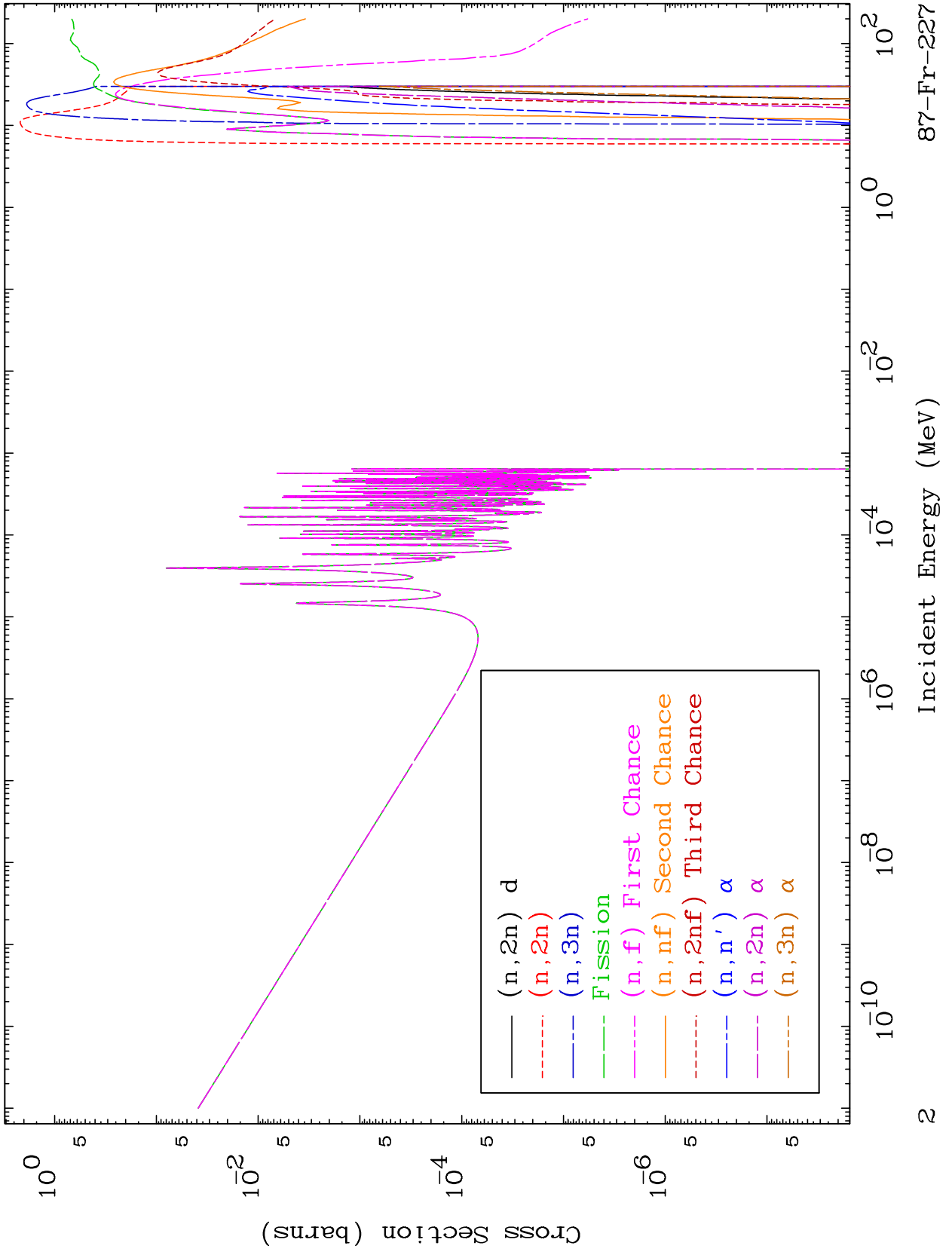


87-Fr-227

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Neutron Absorption  
293 Kelvin Cross Sections

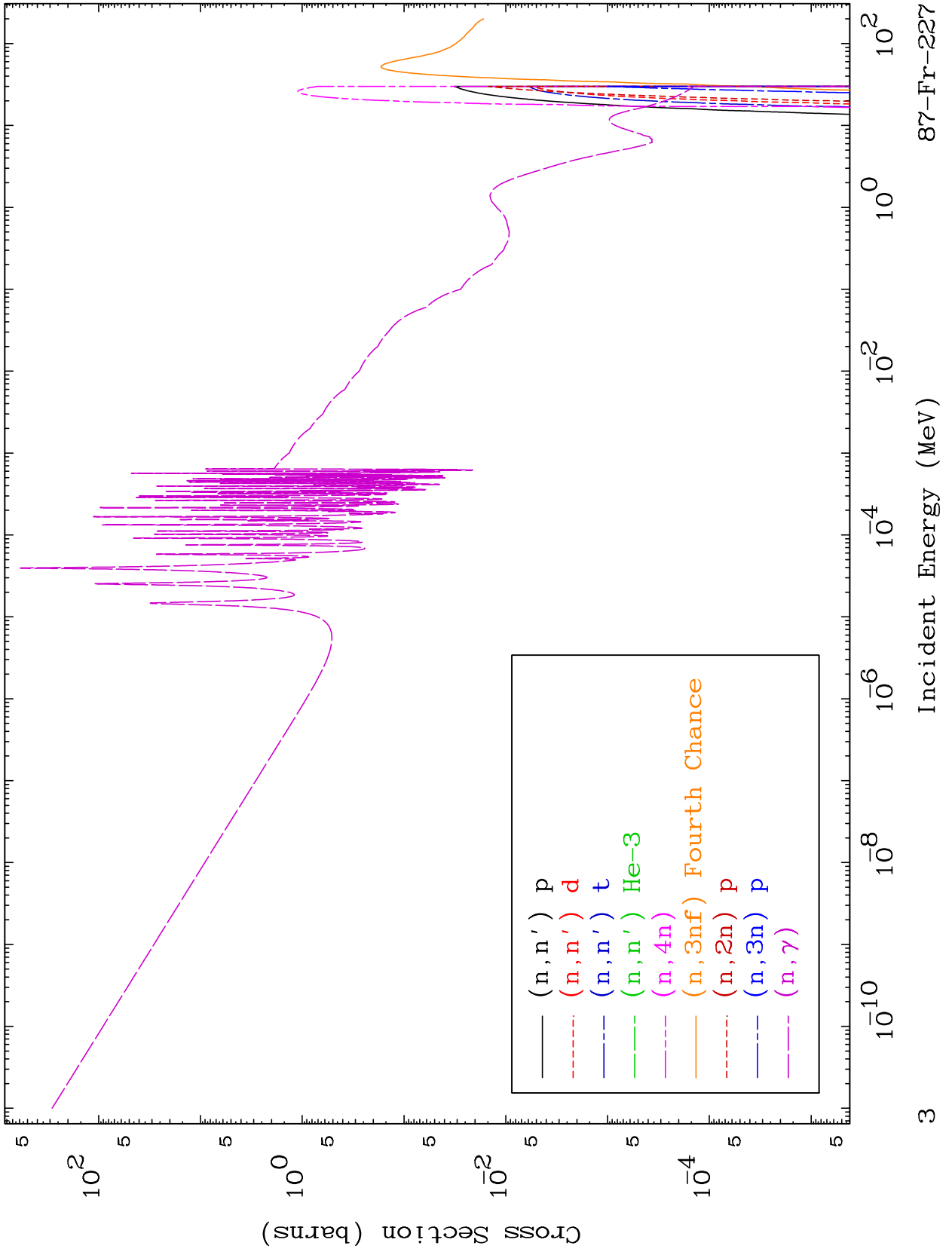
87-Fr-227



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Neutron Absorption  
293 Kelvin Cross Sections

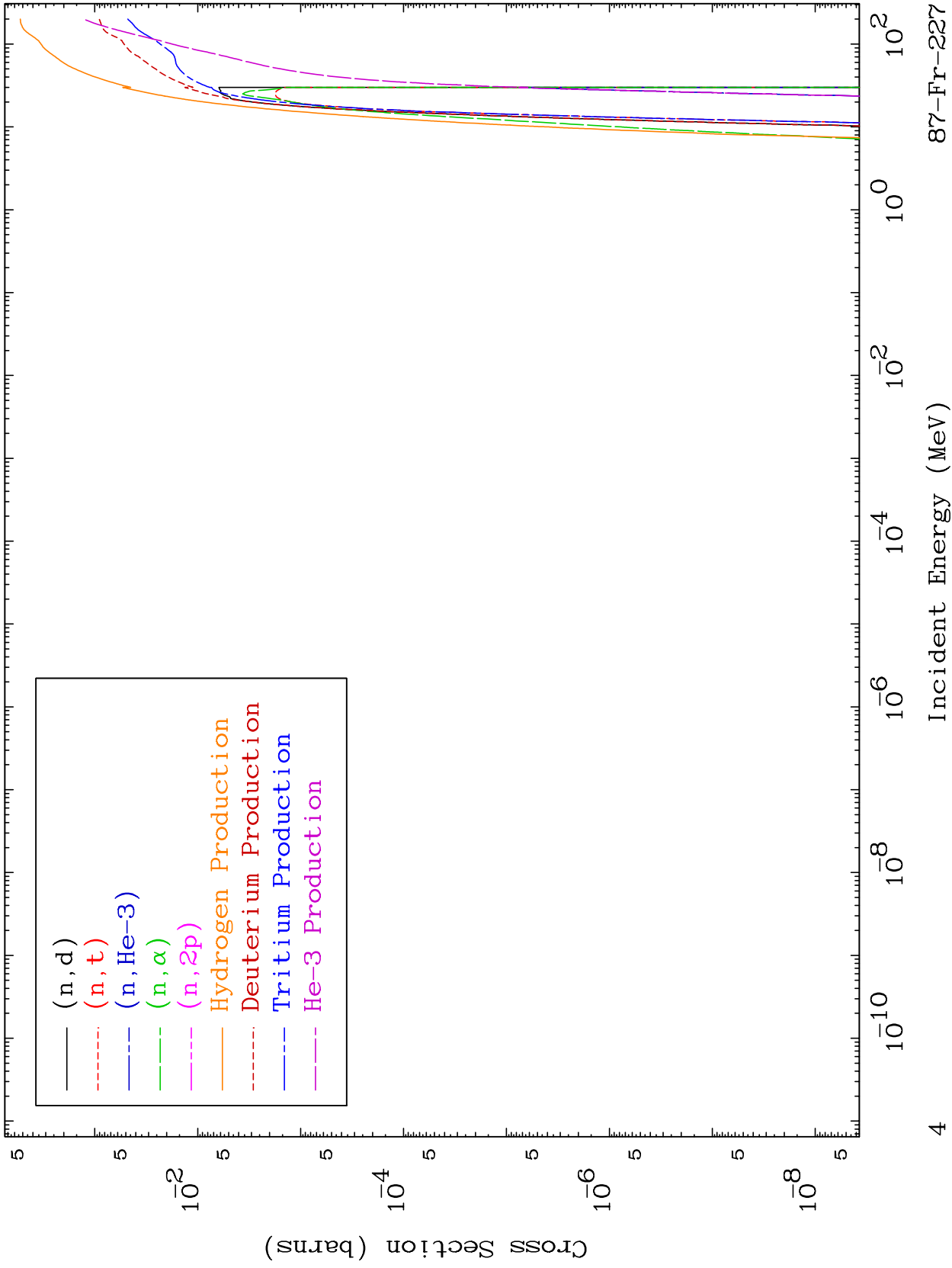
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Neutron Absorption  
293 Kelvin Cross Sections

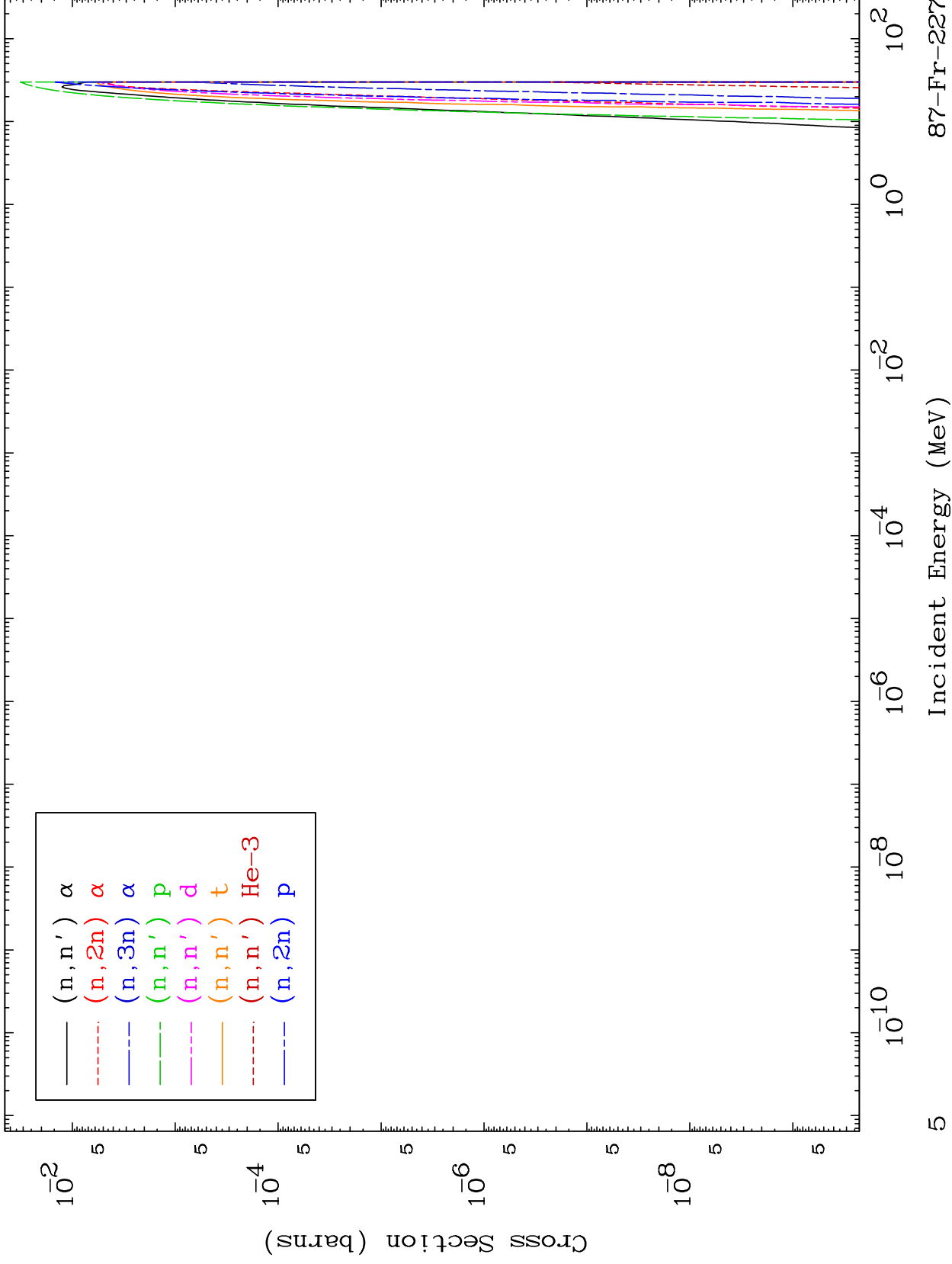
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Charged Particle  
293 Kelvin Cross Sections

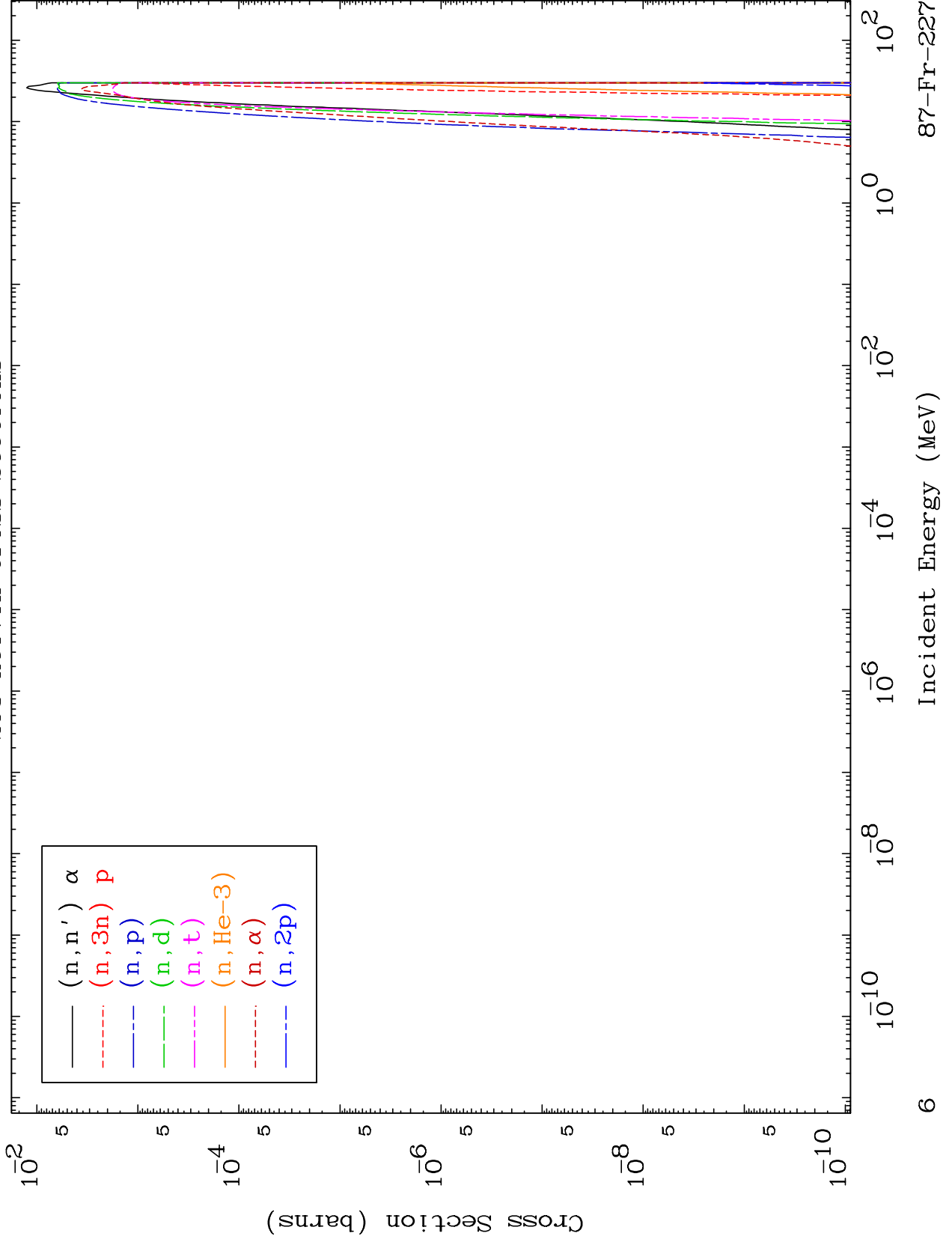
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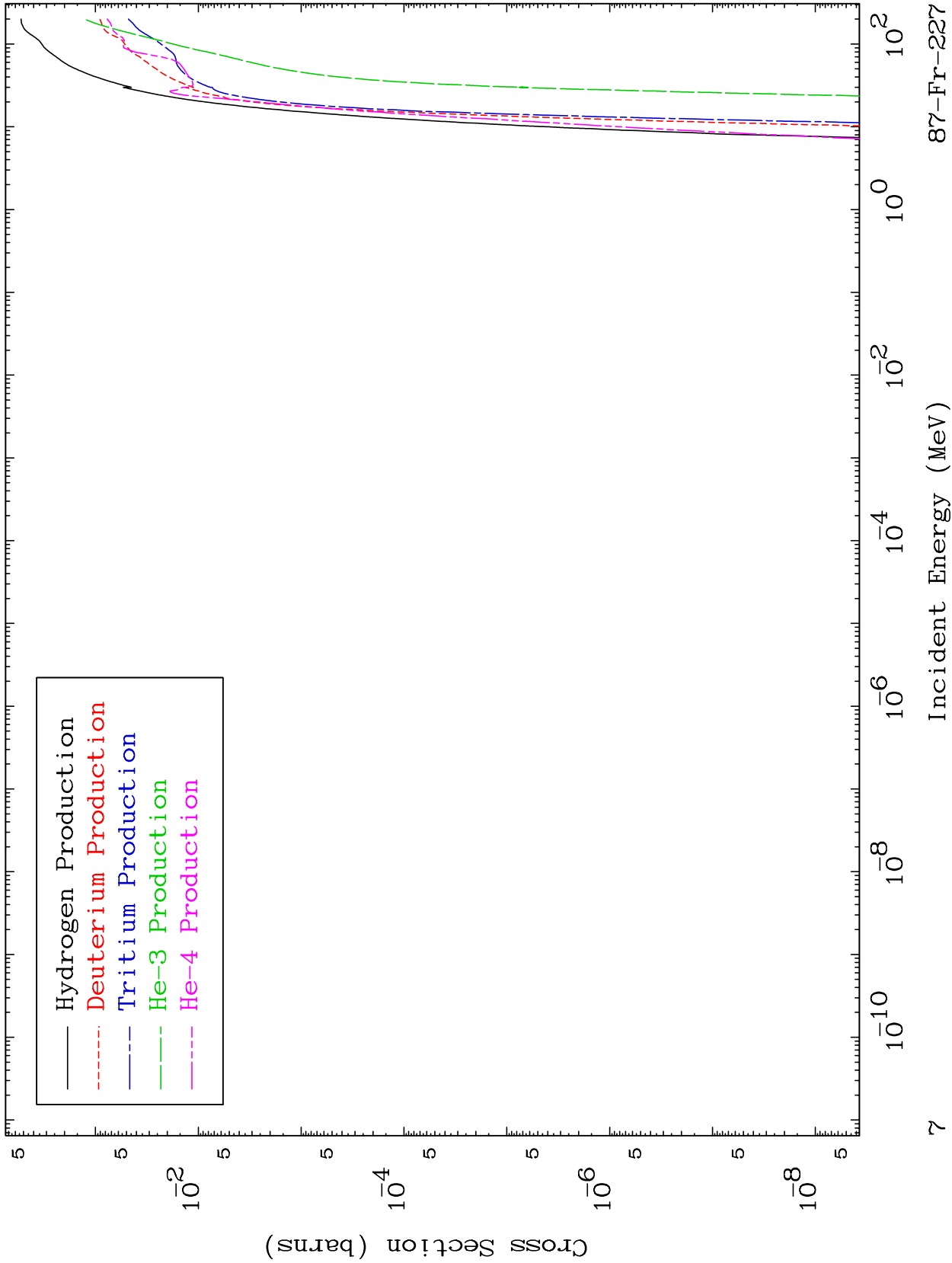


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Charged Particle  
293 Kelvin Cross Sections

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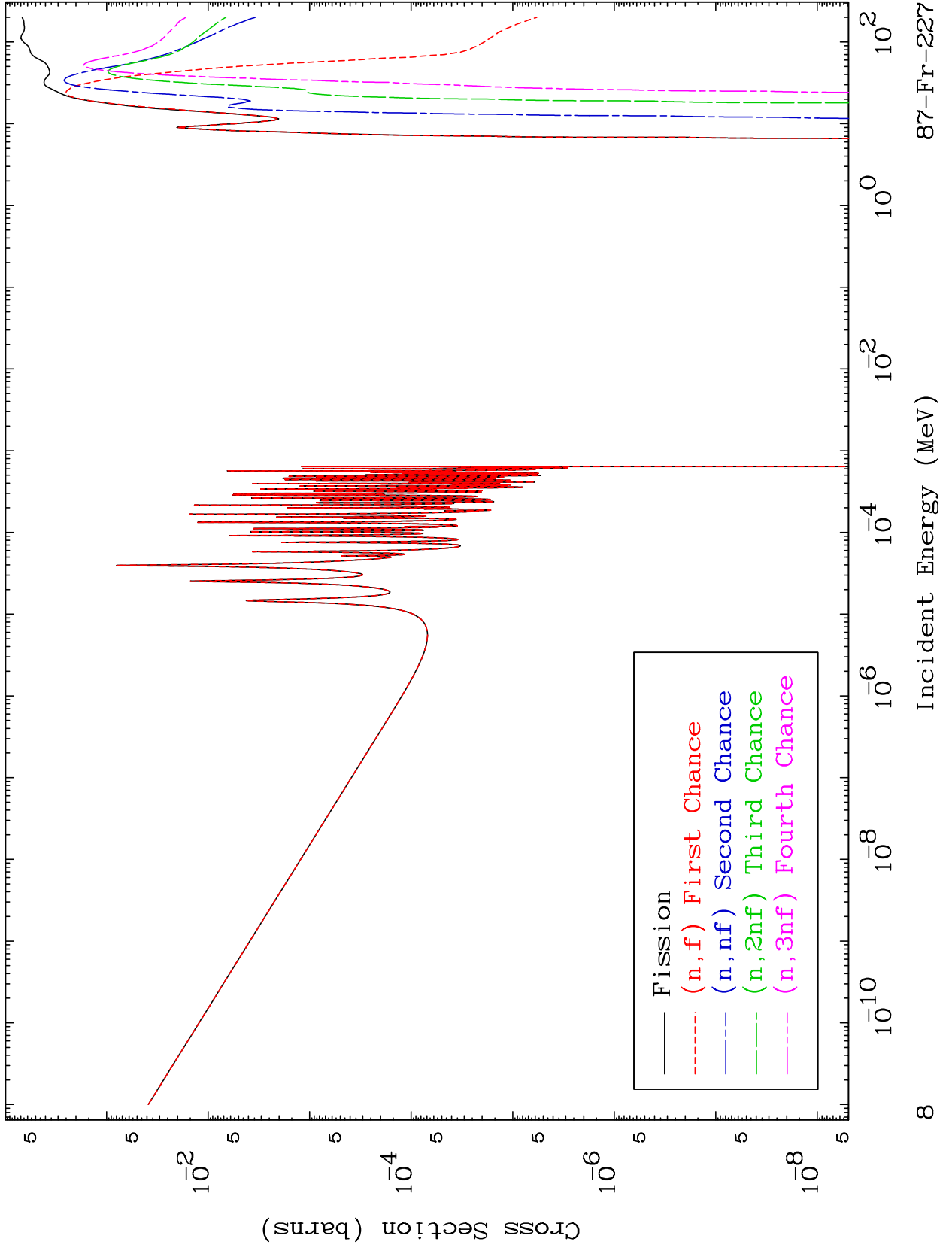


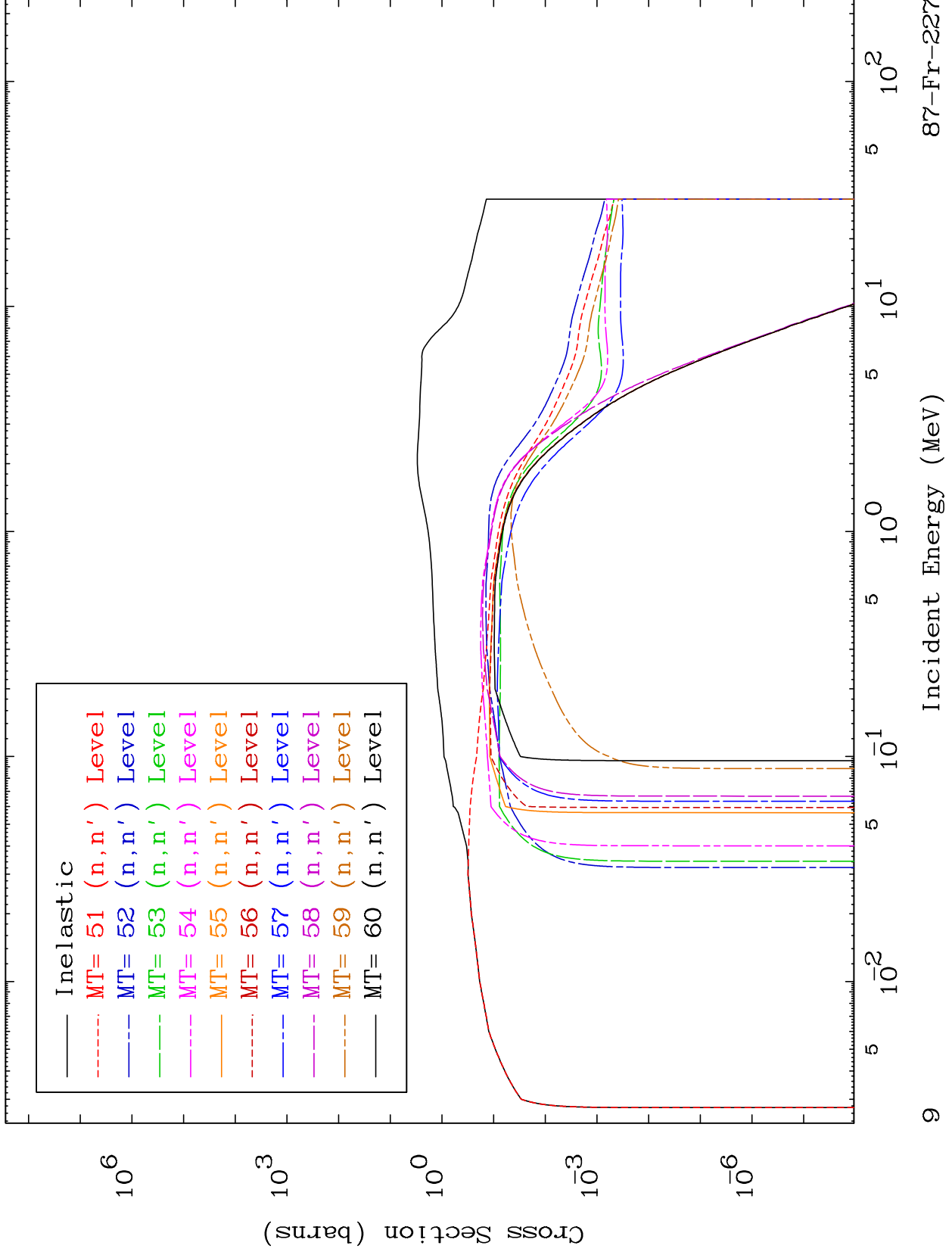


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Fission  
293 Kelvin Cross Sections

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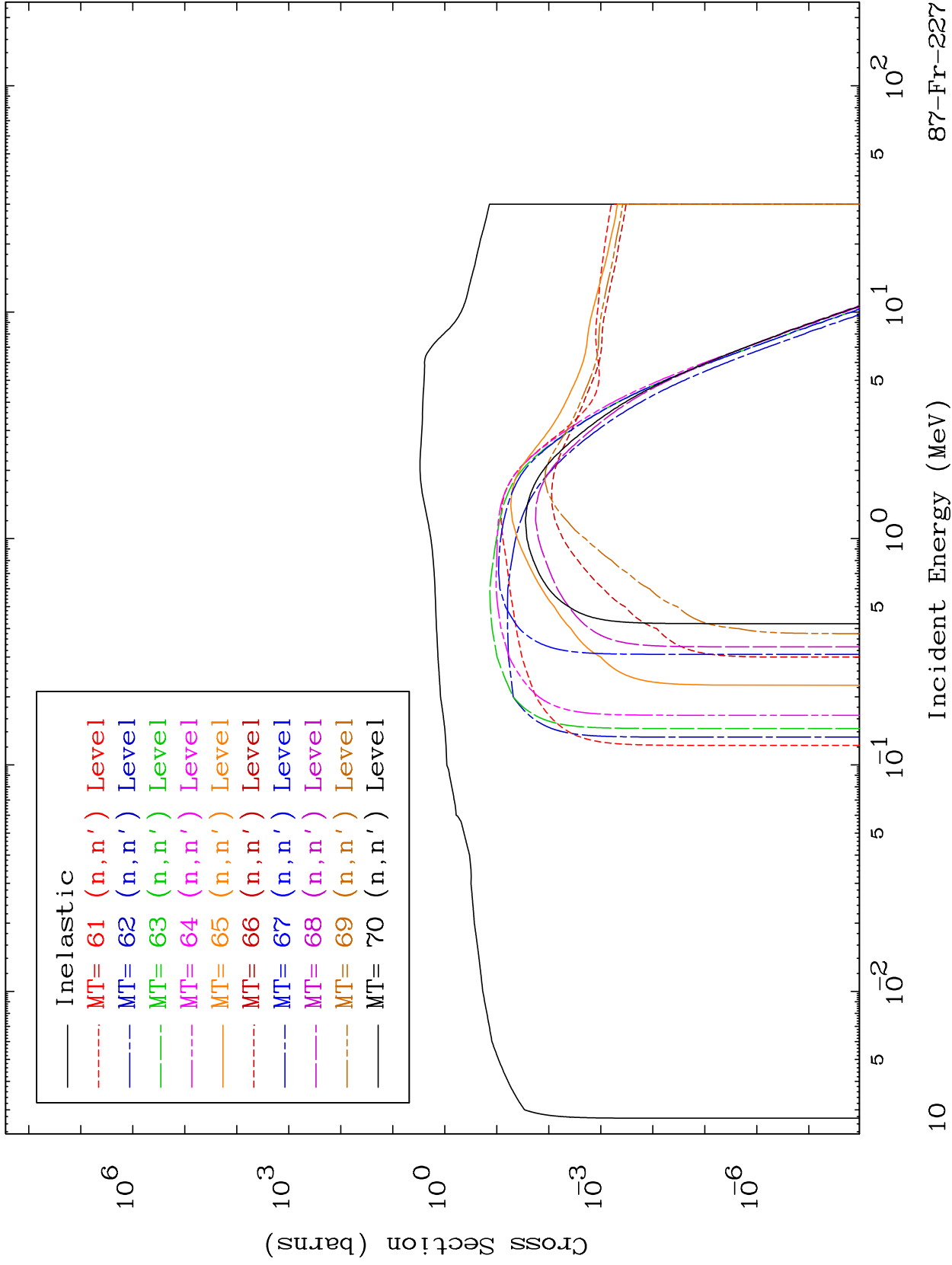




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(n,n') Levels  
293 Kelvin Cross Sections

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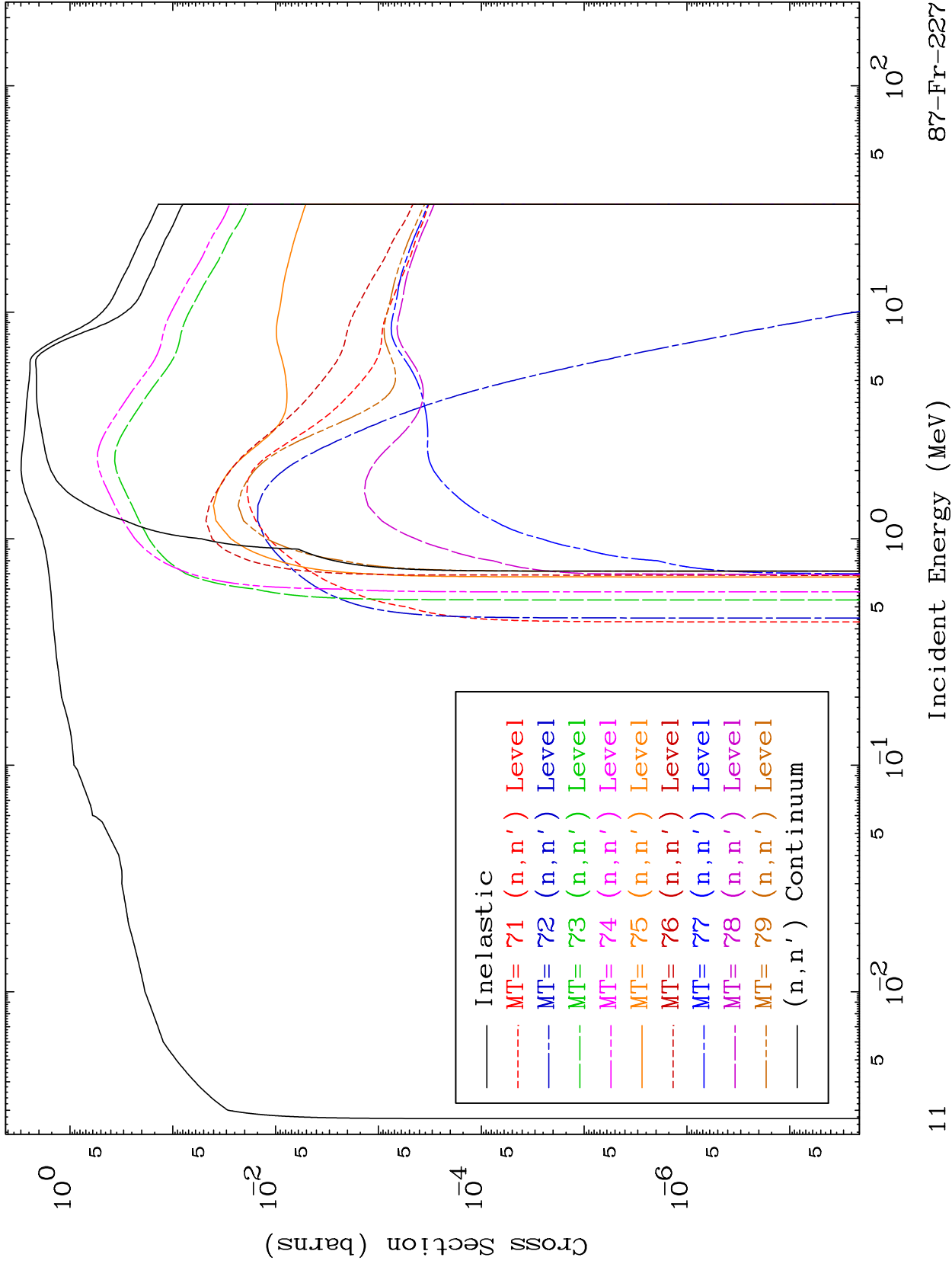
Incident Energy (MeV)

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293 Kelvin Cross Sections  
(n, n') Levels

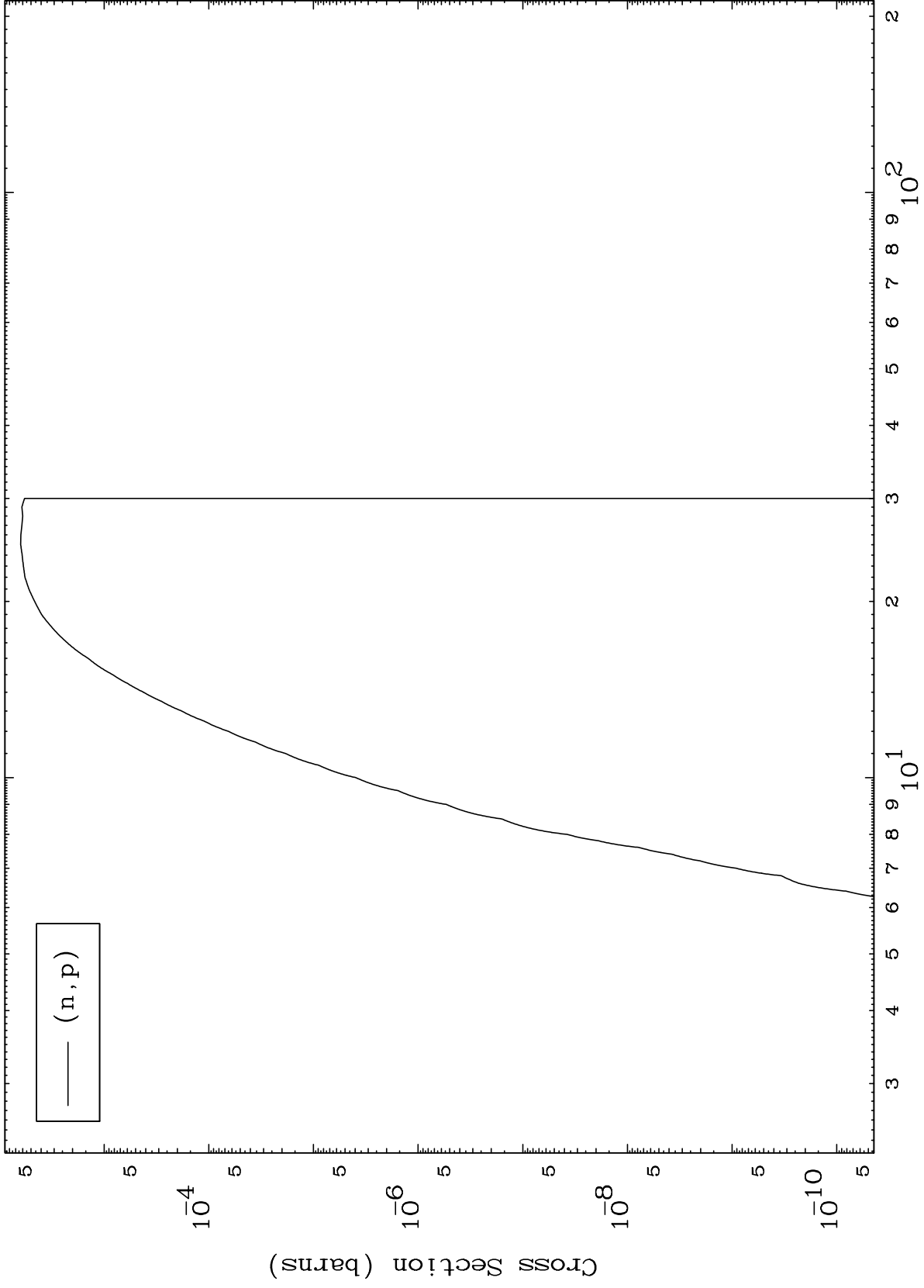
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(n,p) Levels  
293 Kelvin Cross Sections

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(n,p)

Incident Energy (MeV)

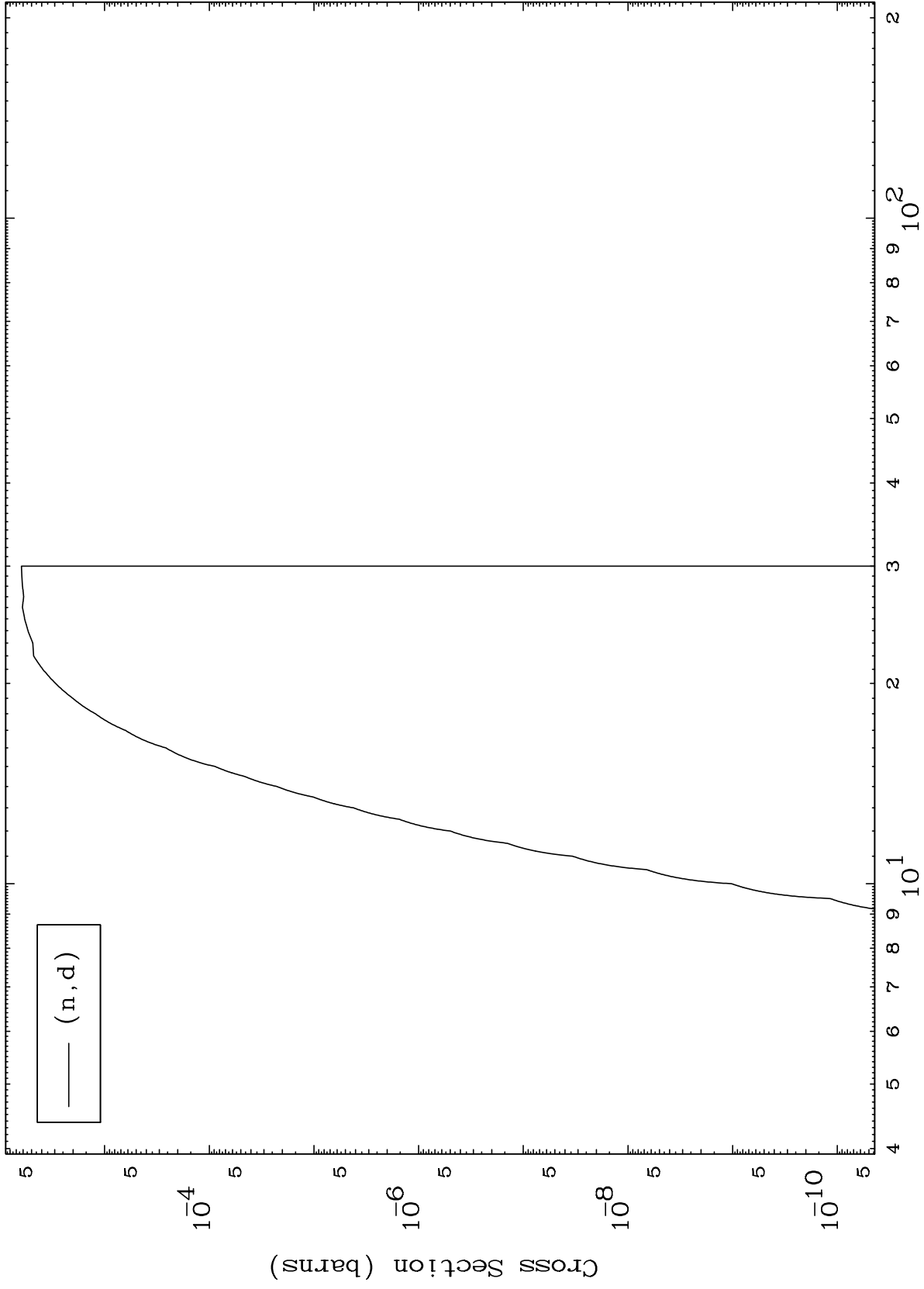
87-Fr-227

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(n,d) Levels  
293 Kelvin Cross Sections

87-Fr-227



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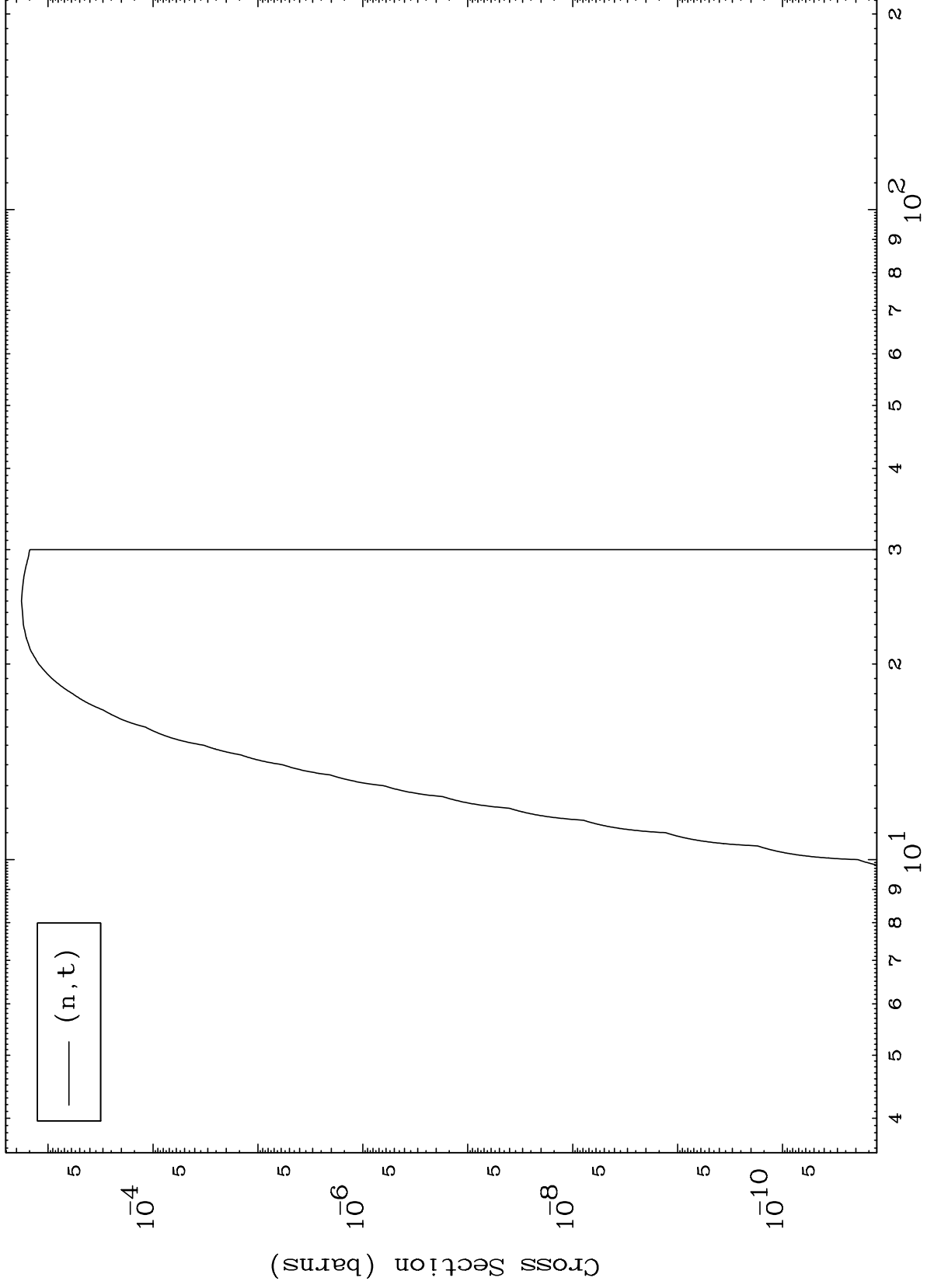
Incident Energy (MeV)

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(n,t) Levels  
293 Kelvin Cross Sections

87-Fr-227



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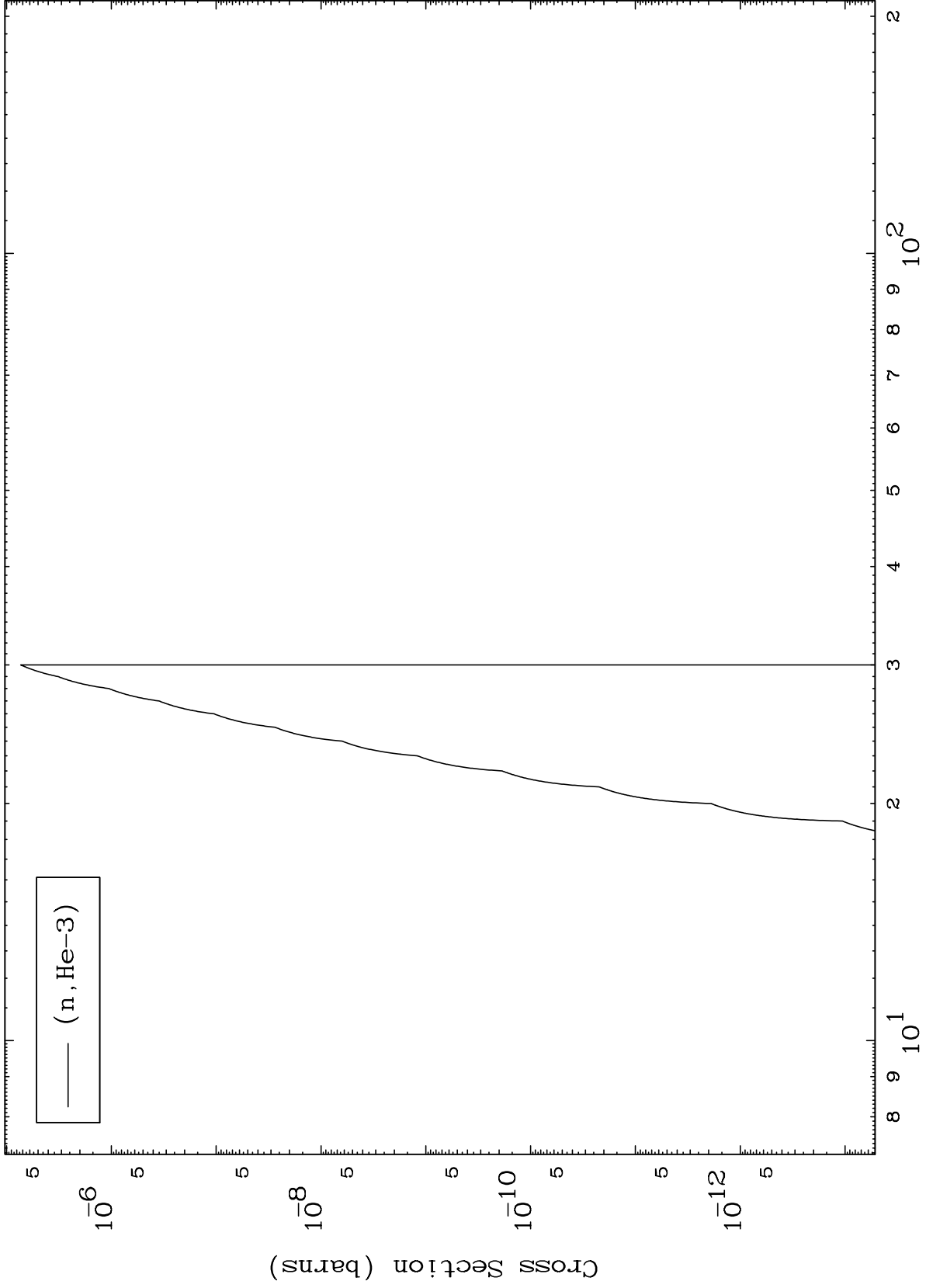
Incident Energy (MeV)

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(n,He3) Levels  
293 Kelvin Cross Sections

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Incident Energy (MeV)

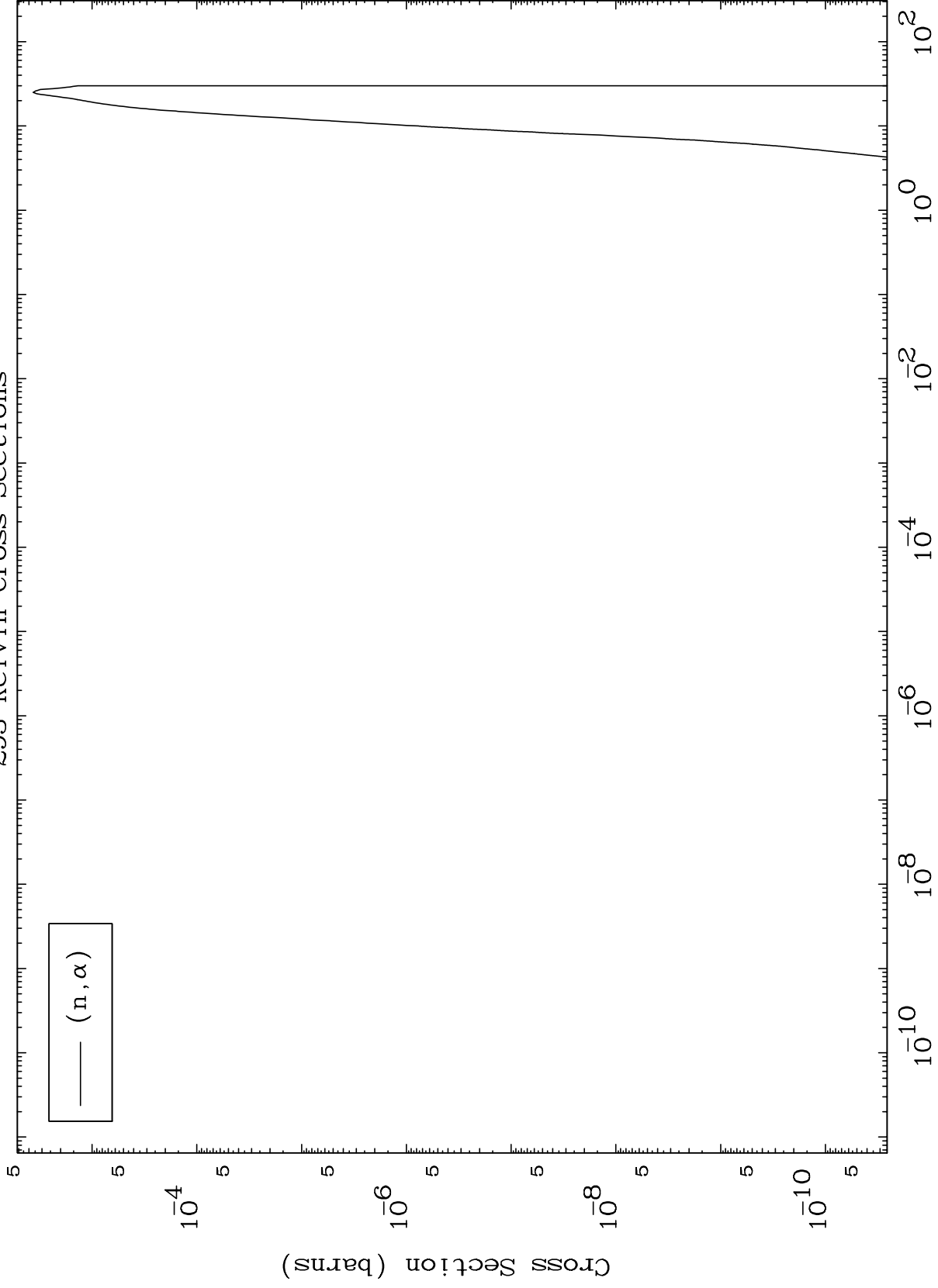
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(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

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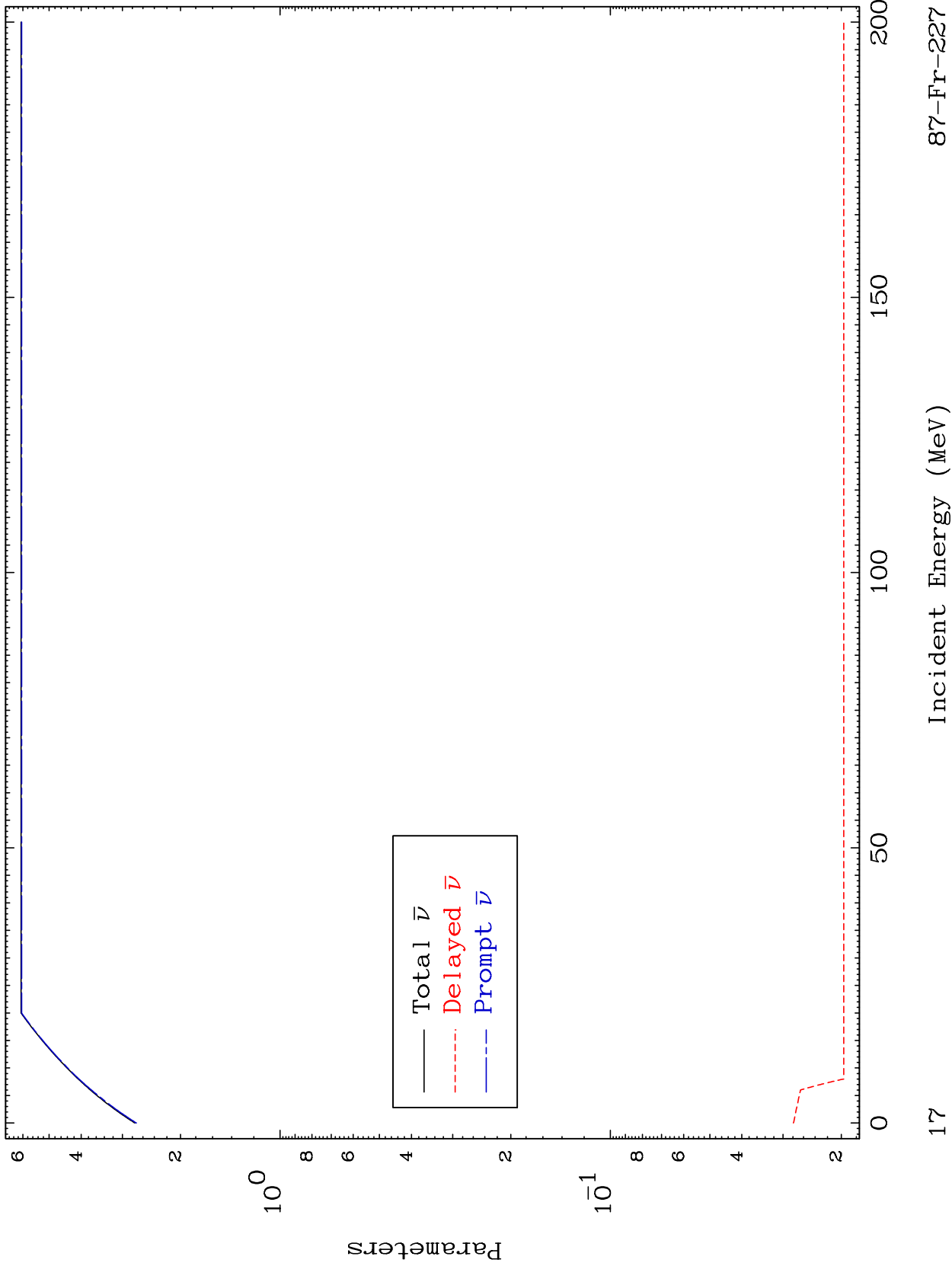


(n,  $\alpha$ )

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Energy Release  
Parameters

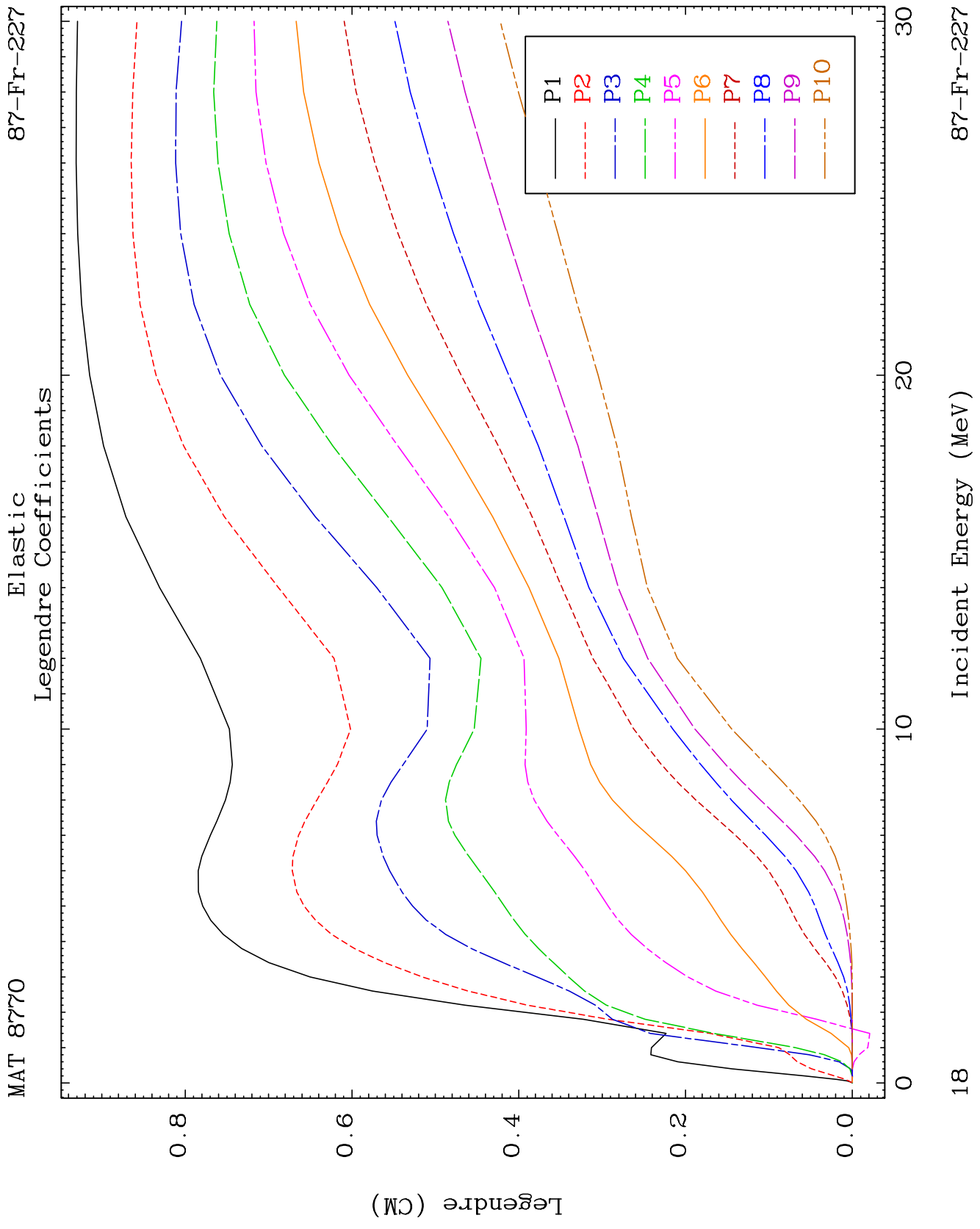
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Incident Energy (MeV)

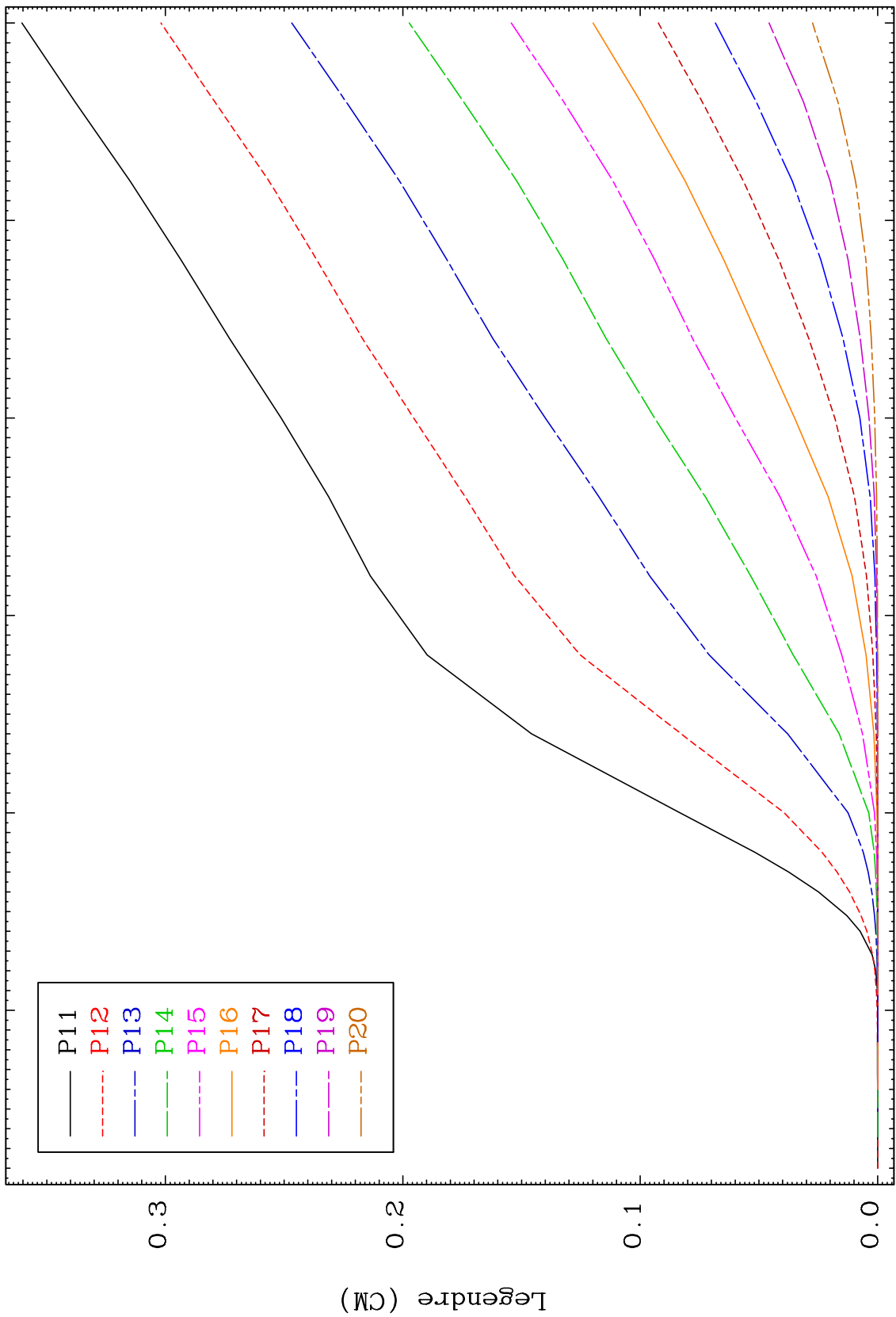
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Elastic Legendre Coefficients

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Incident Energy (MeV)

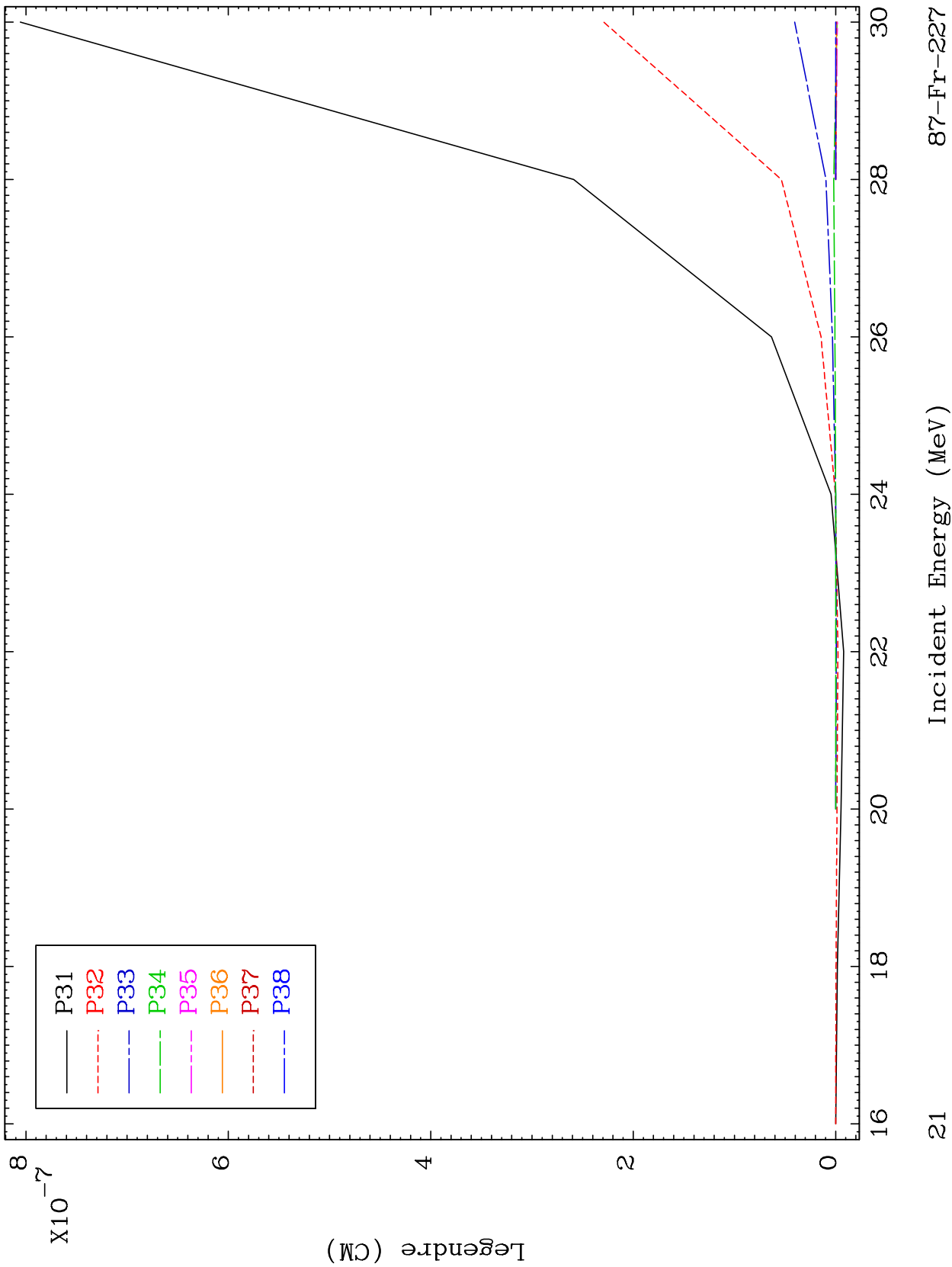
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Elastic  
Legendre Coefficients

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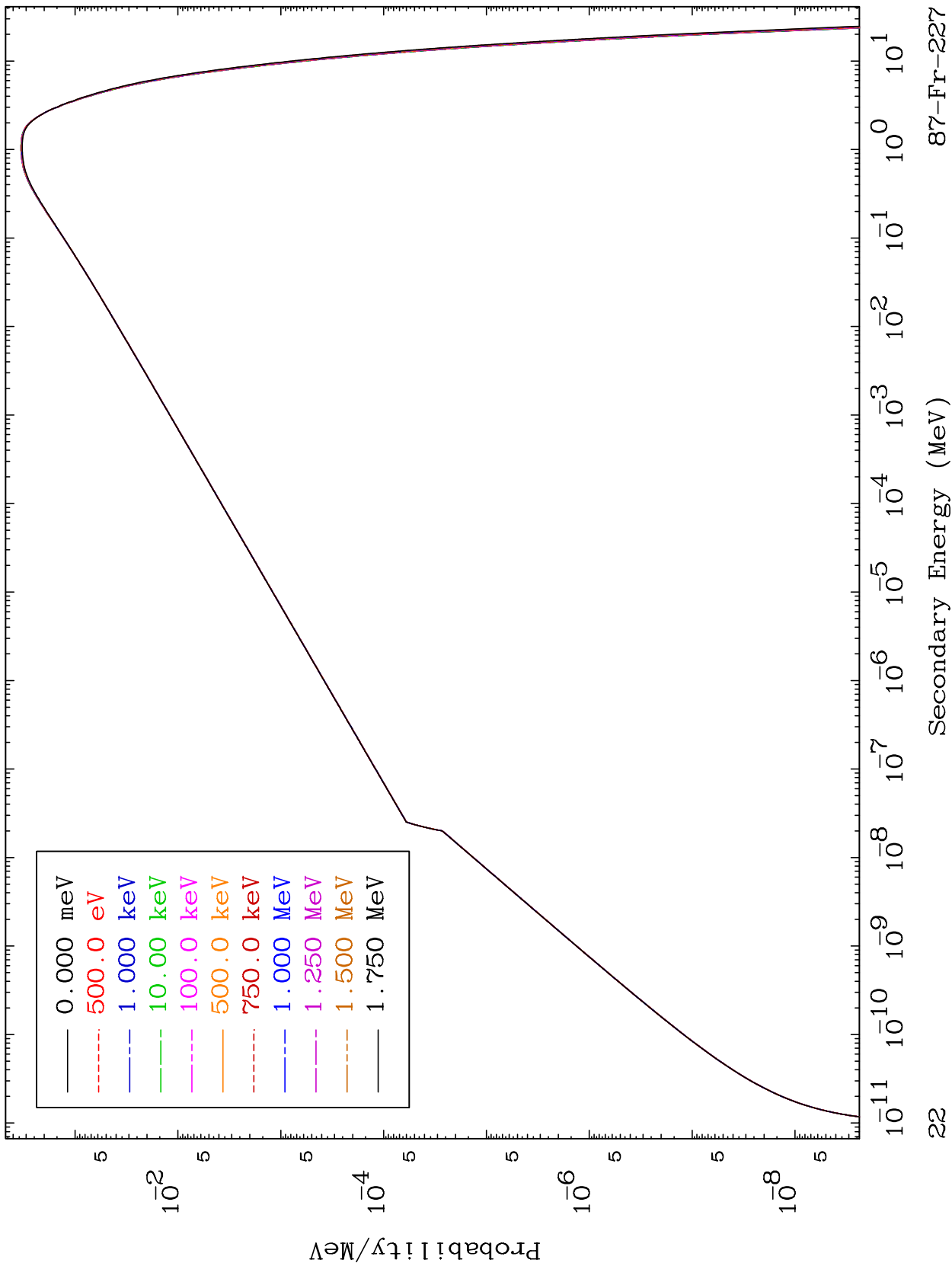
Incident Energy (MeV)

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### Fission Energy Distributions

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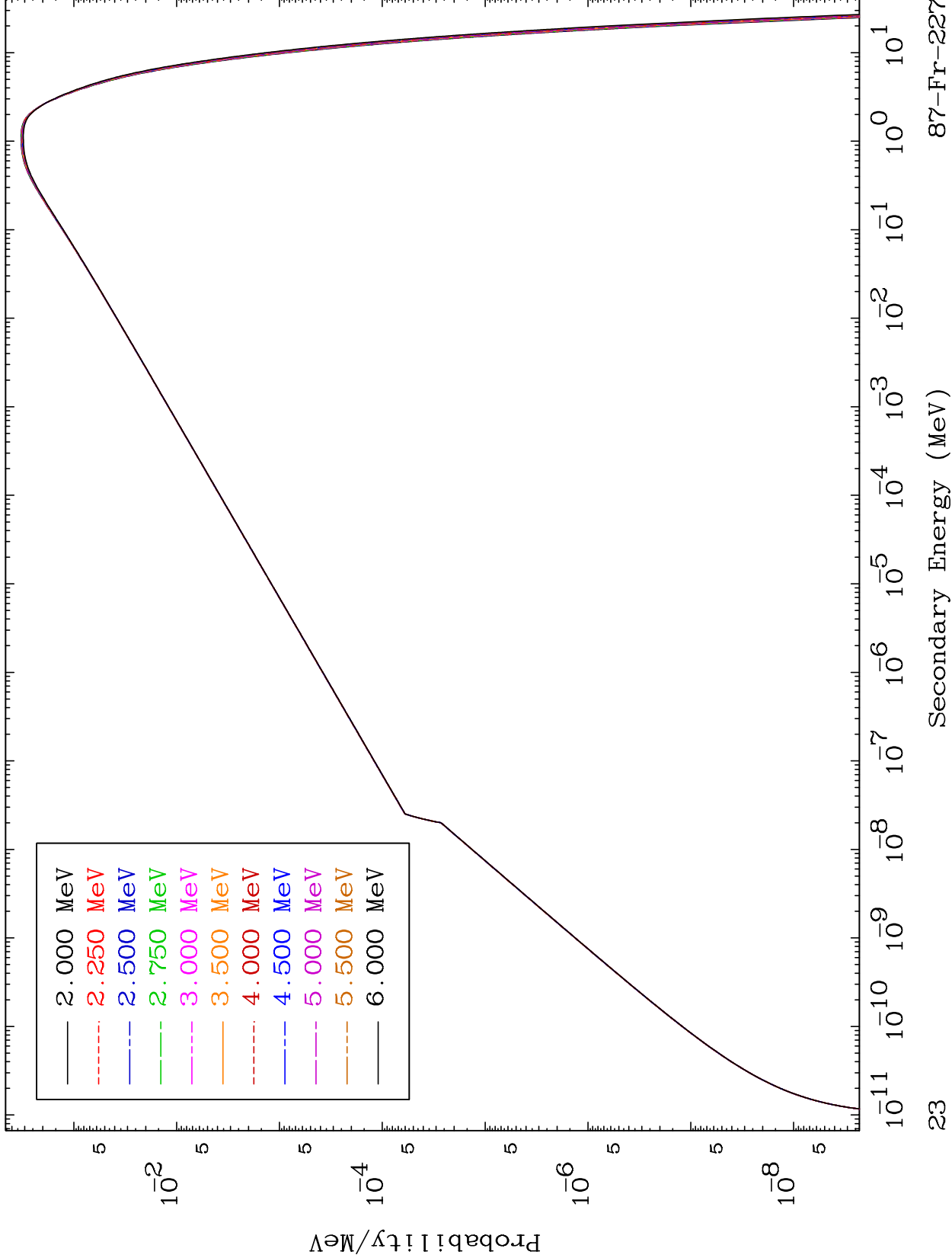
Secondary Energy (MeV)

22

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### Fission Energy Distributions

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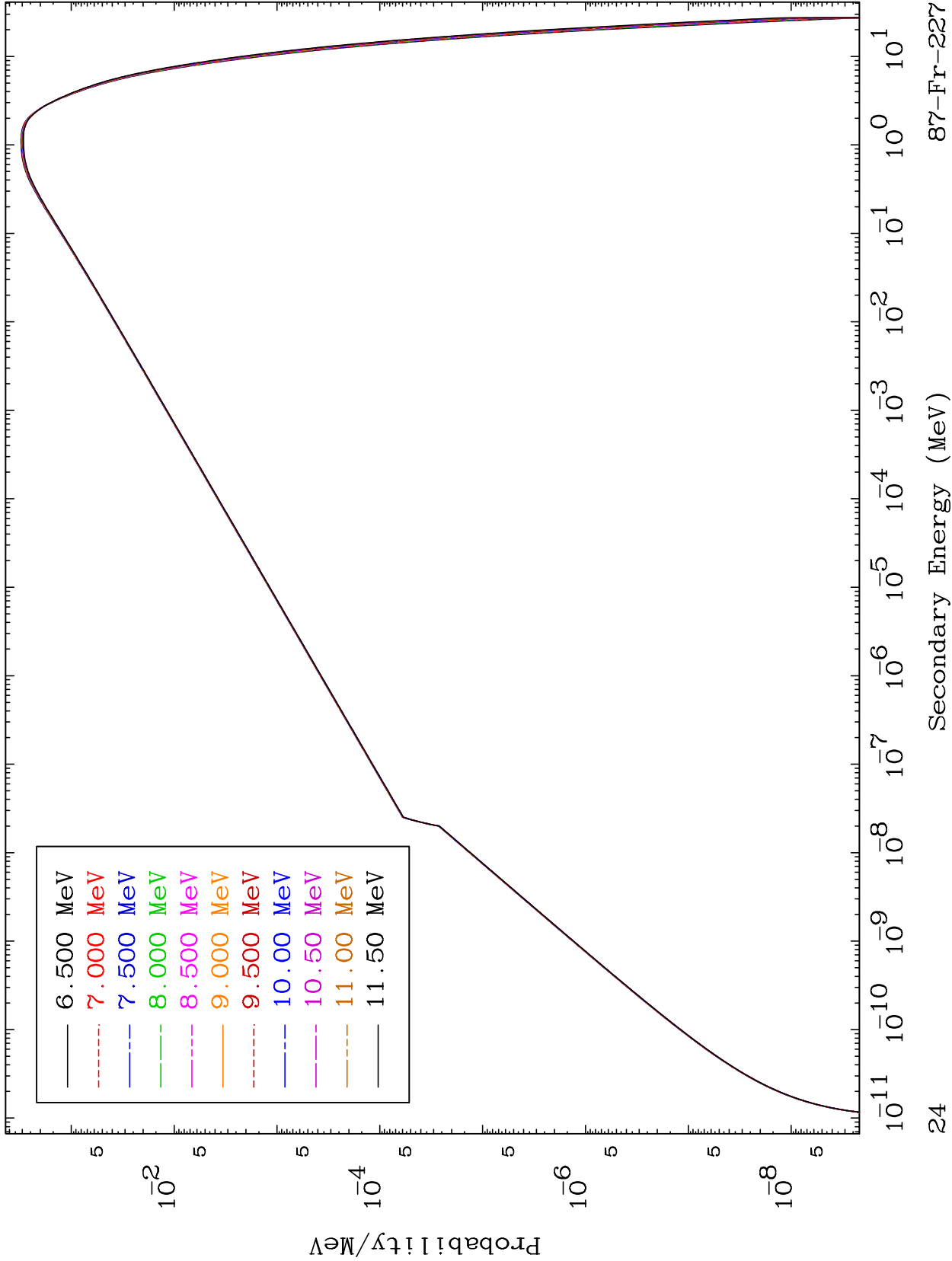




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### Fission Energy Distributions

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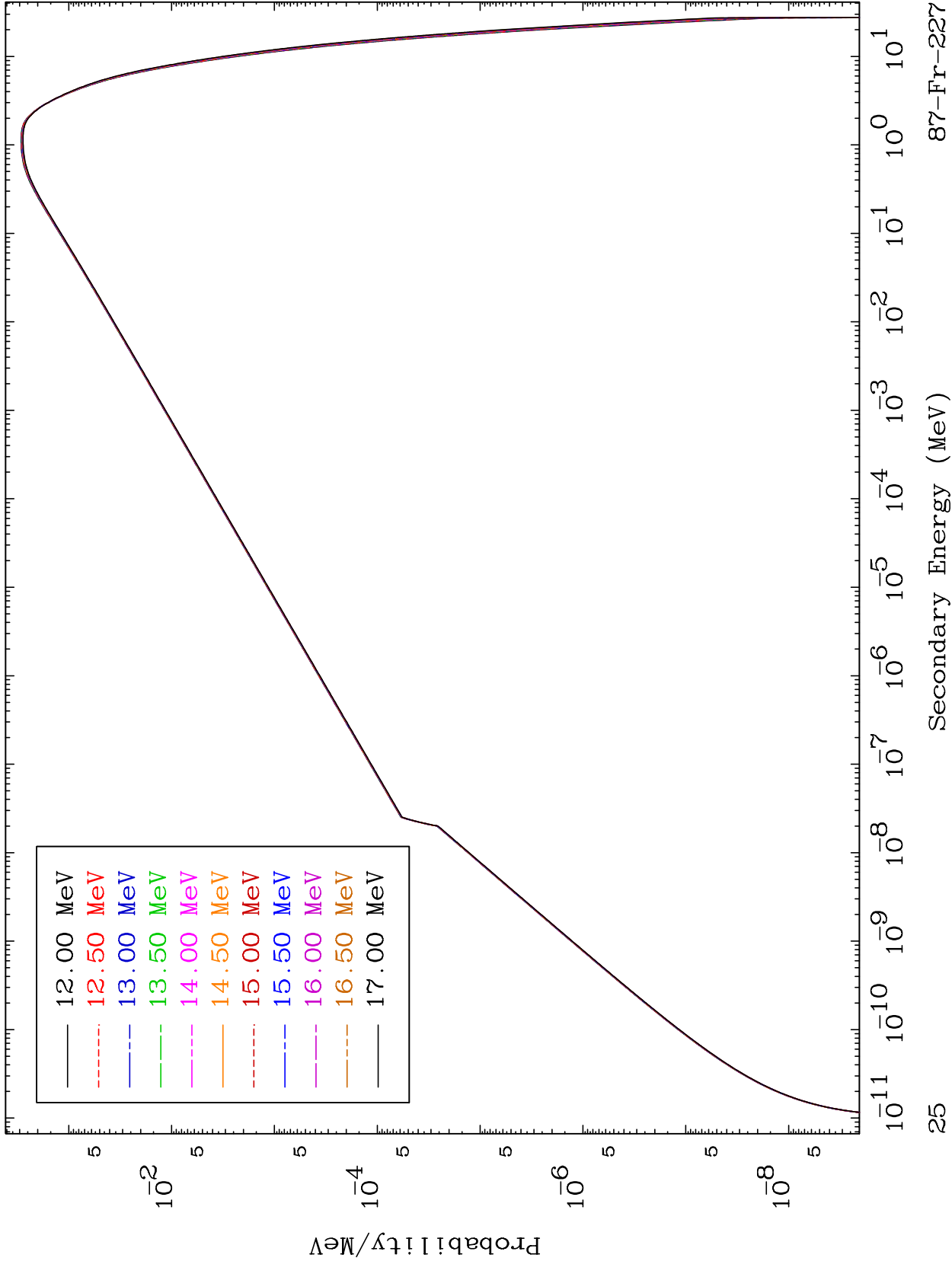
Secondary Energy (MeV)

24

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### Fission Energy Distributions

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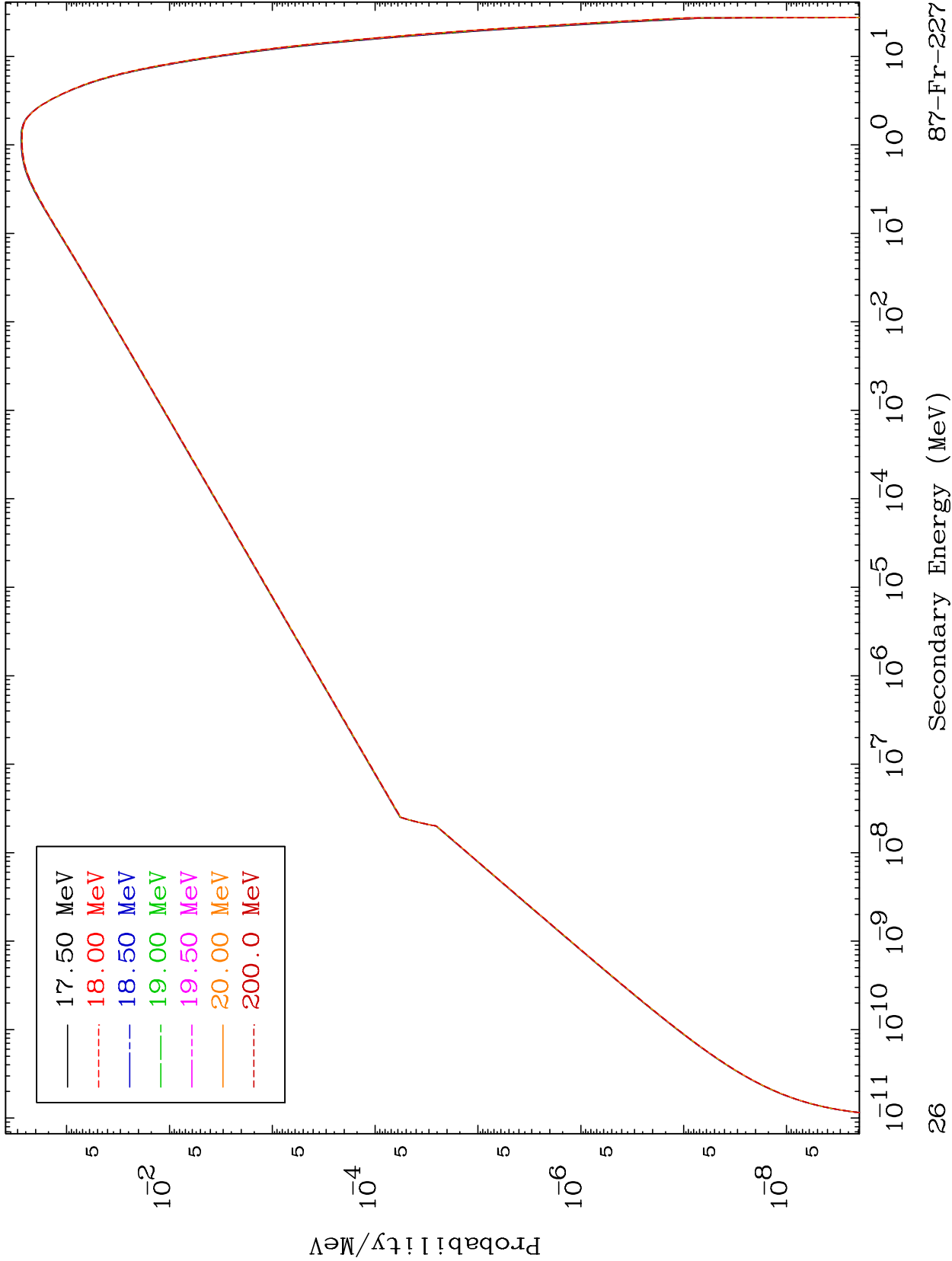


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Secondary Energy (MeV)

25

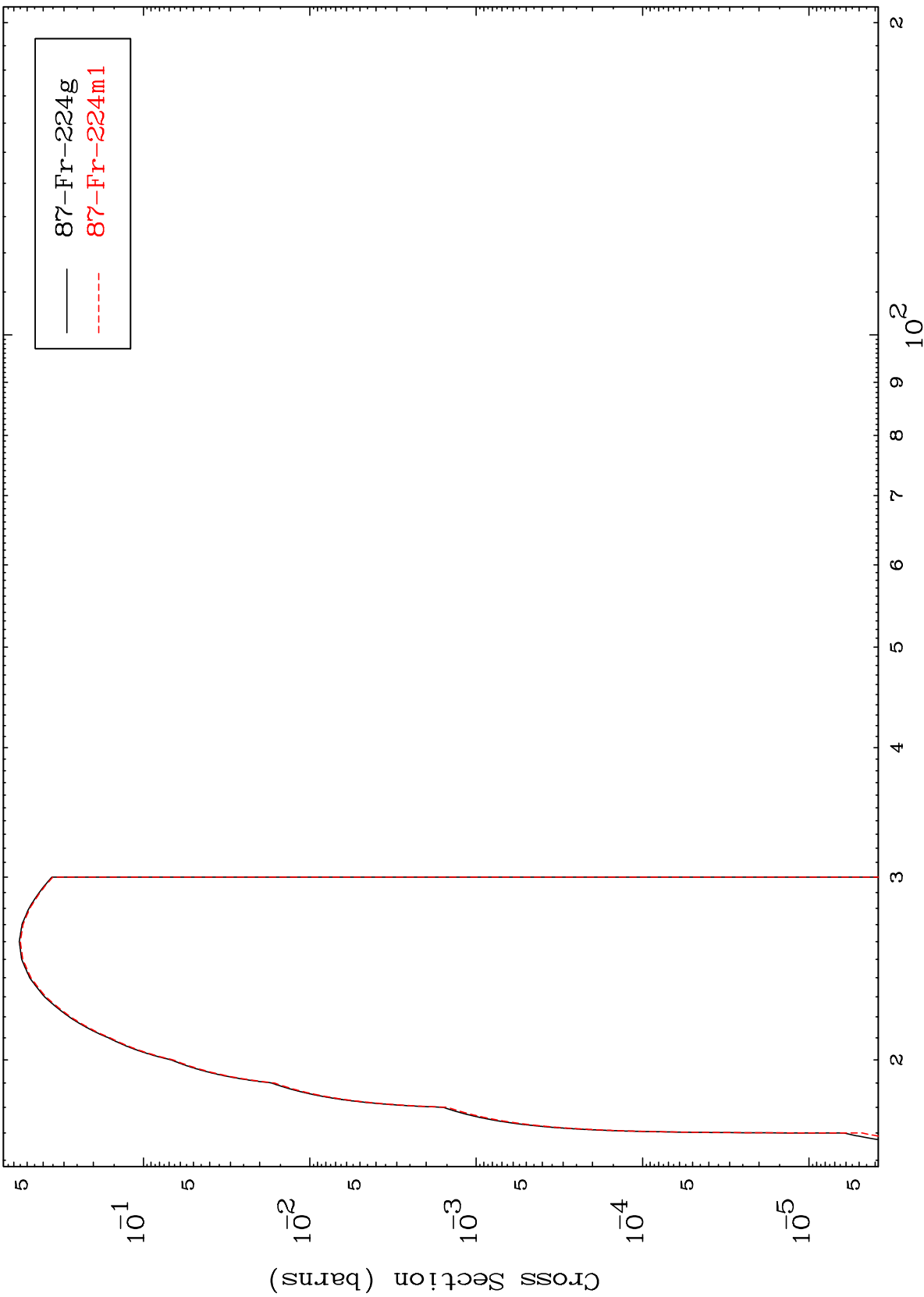
Fission  
Energy Distributions



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(n,4n)  
Radionuclide Production Cross Section



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Incident Energy (MeV)

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