

Program EVALPLOT
(Version 2021-1)

by

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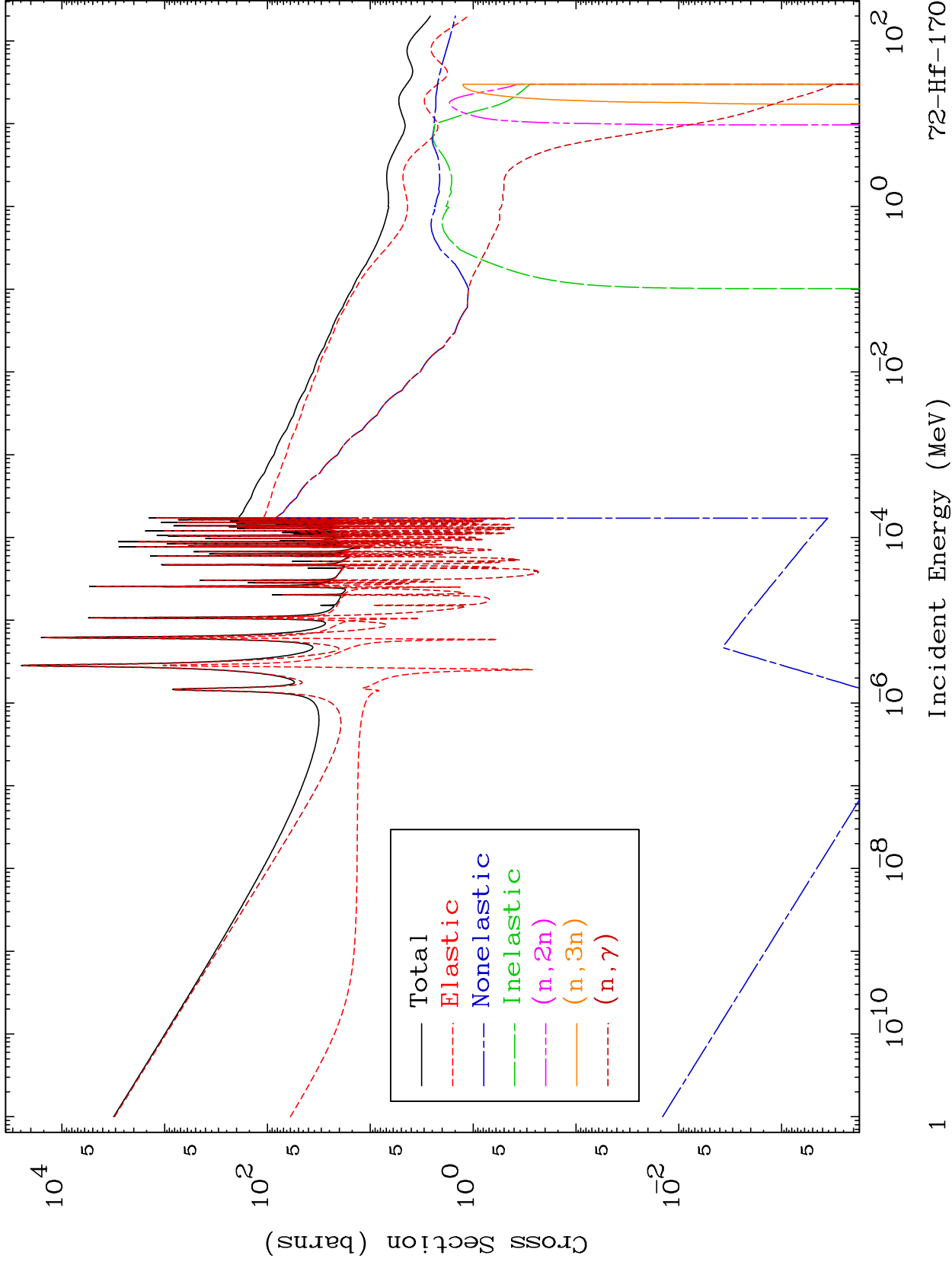
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 7213

Neutron Major
293 Kelvin Cross Sections

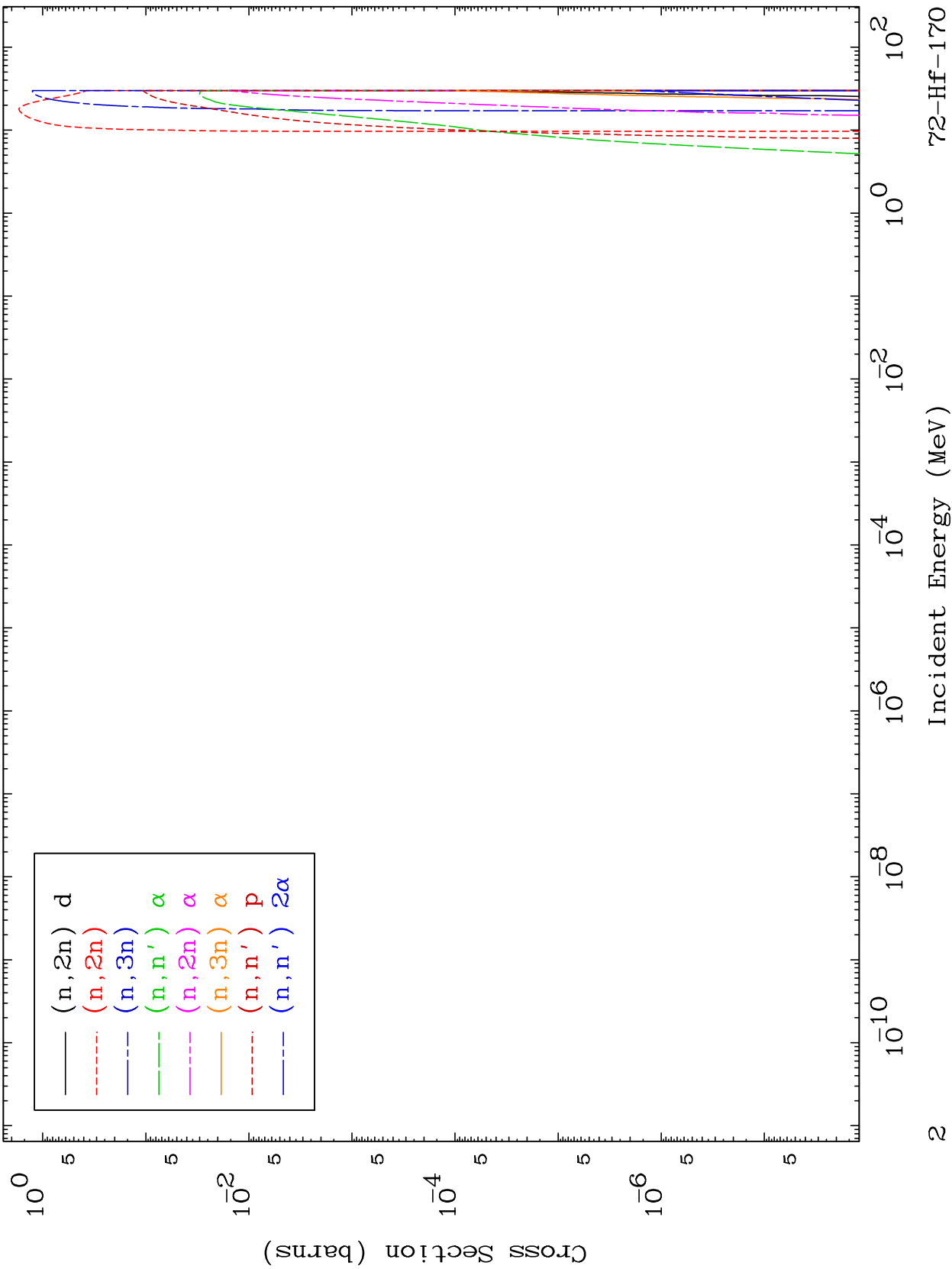
72-Hf-170



MAT 7213

Neutron Absorption
293 Kelvin Cross Sections

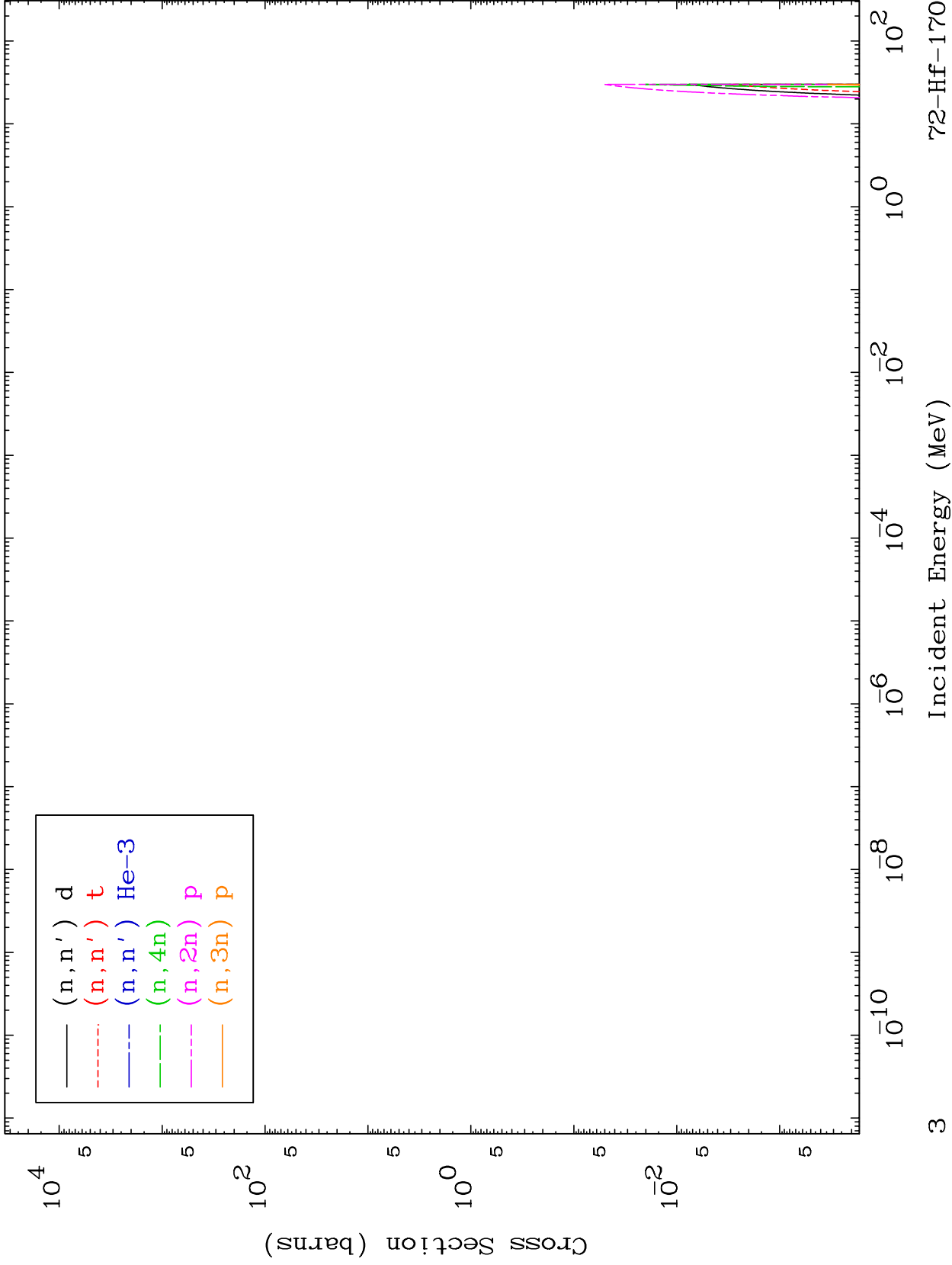
72-Hf-170



MAT 7213

Neutron Absorption
293 Kelvin Cross Sections

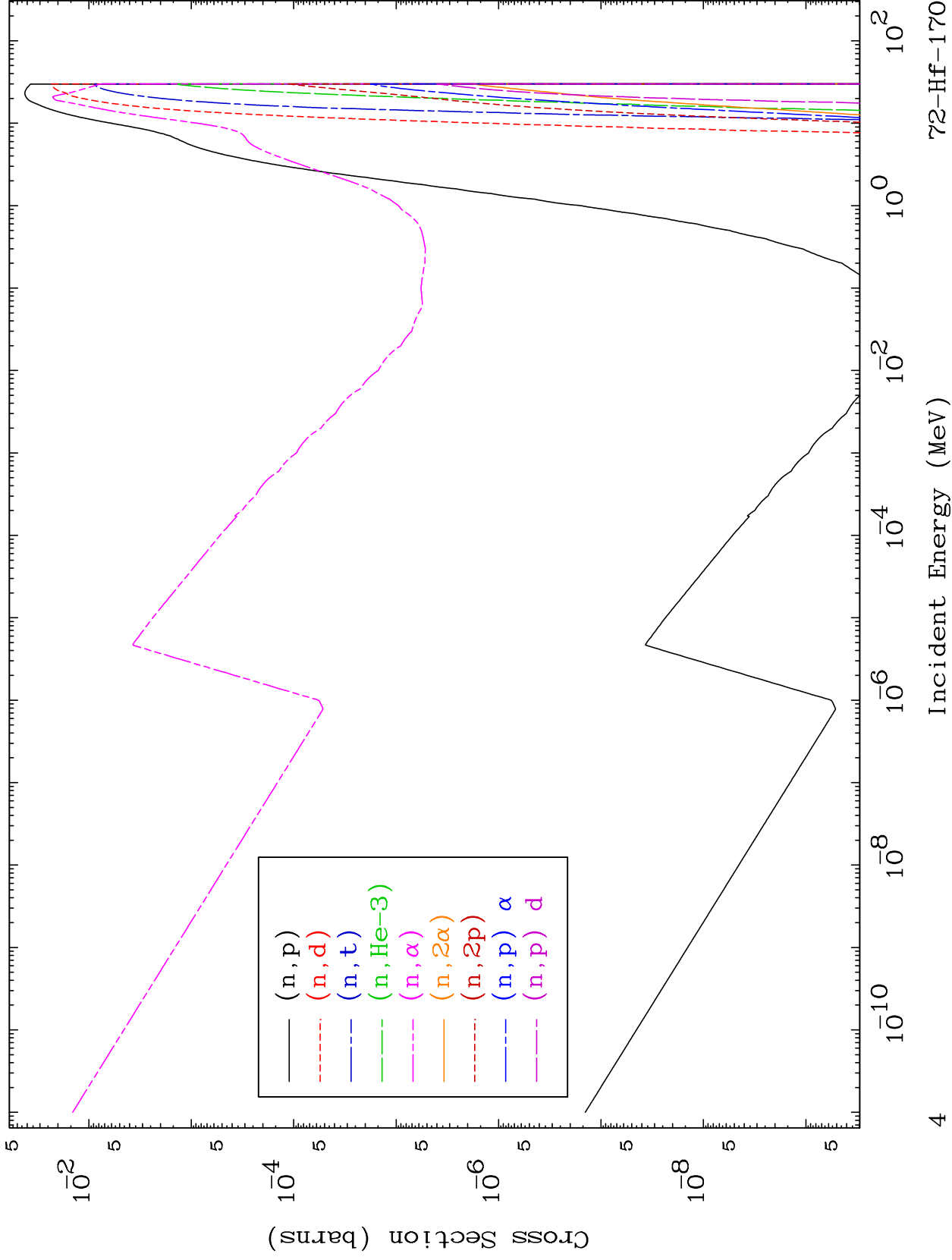
72-Hf-170

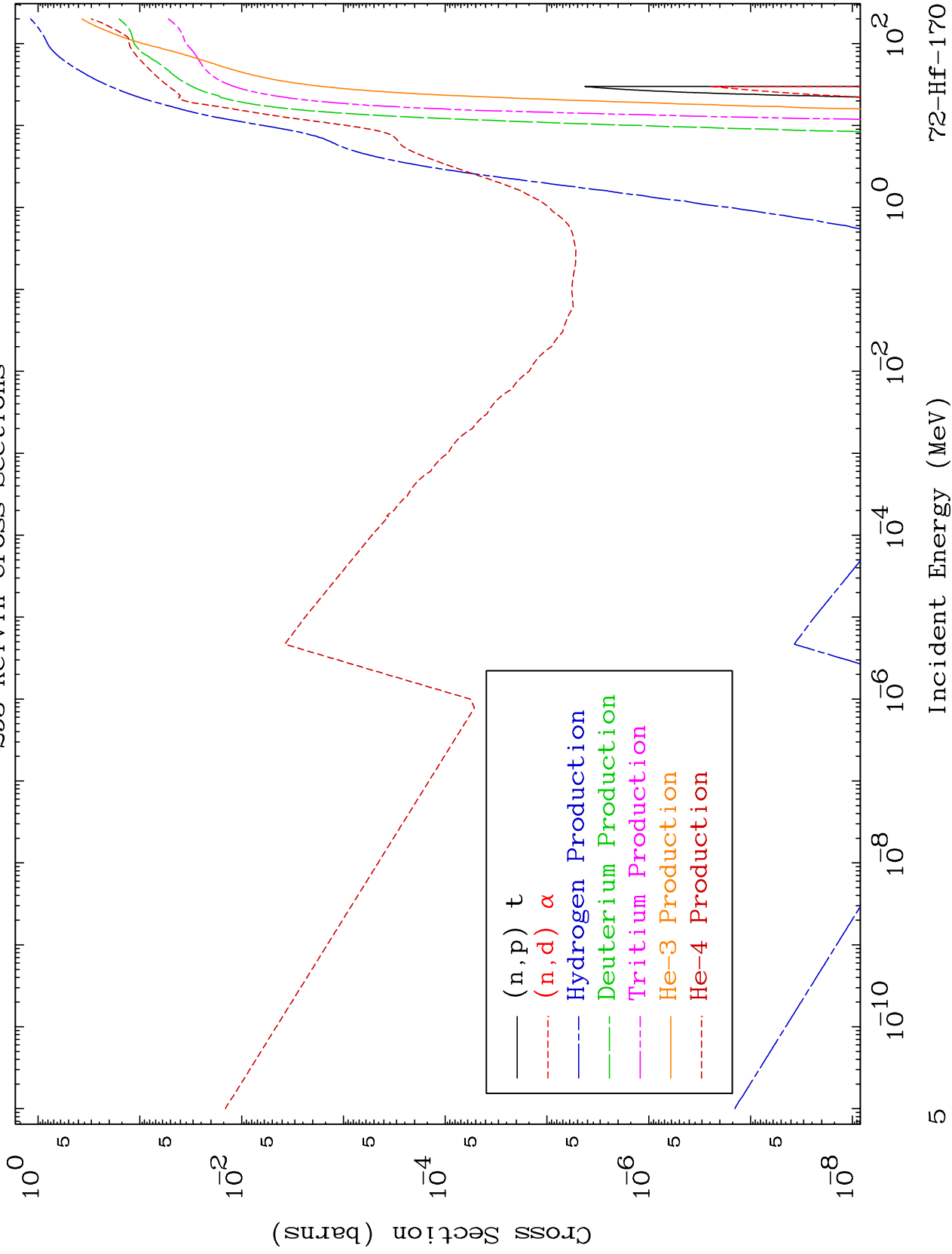


MAT 7213

Neutron Absorption
293 Kelvin Cross Sections

72-Hf-170

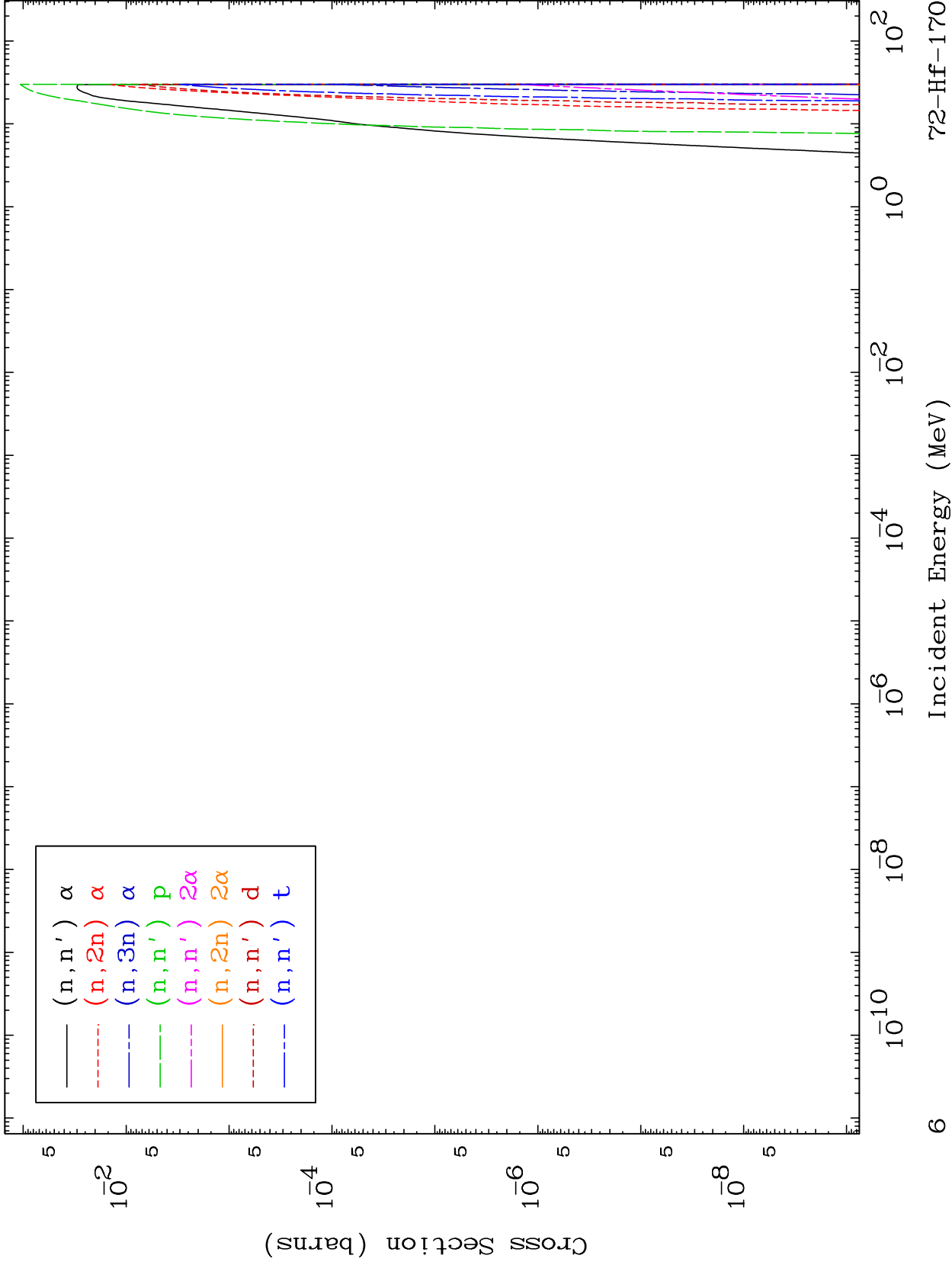




MAT 7213

Charged Particle
293 Kelvin Cross Sections

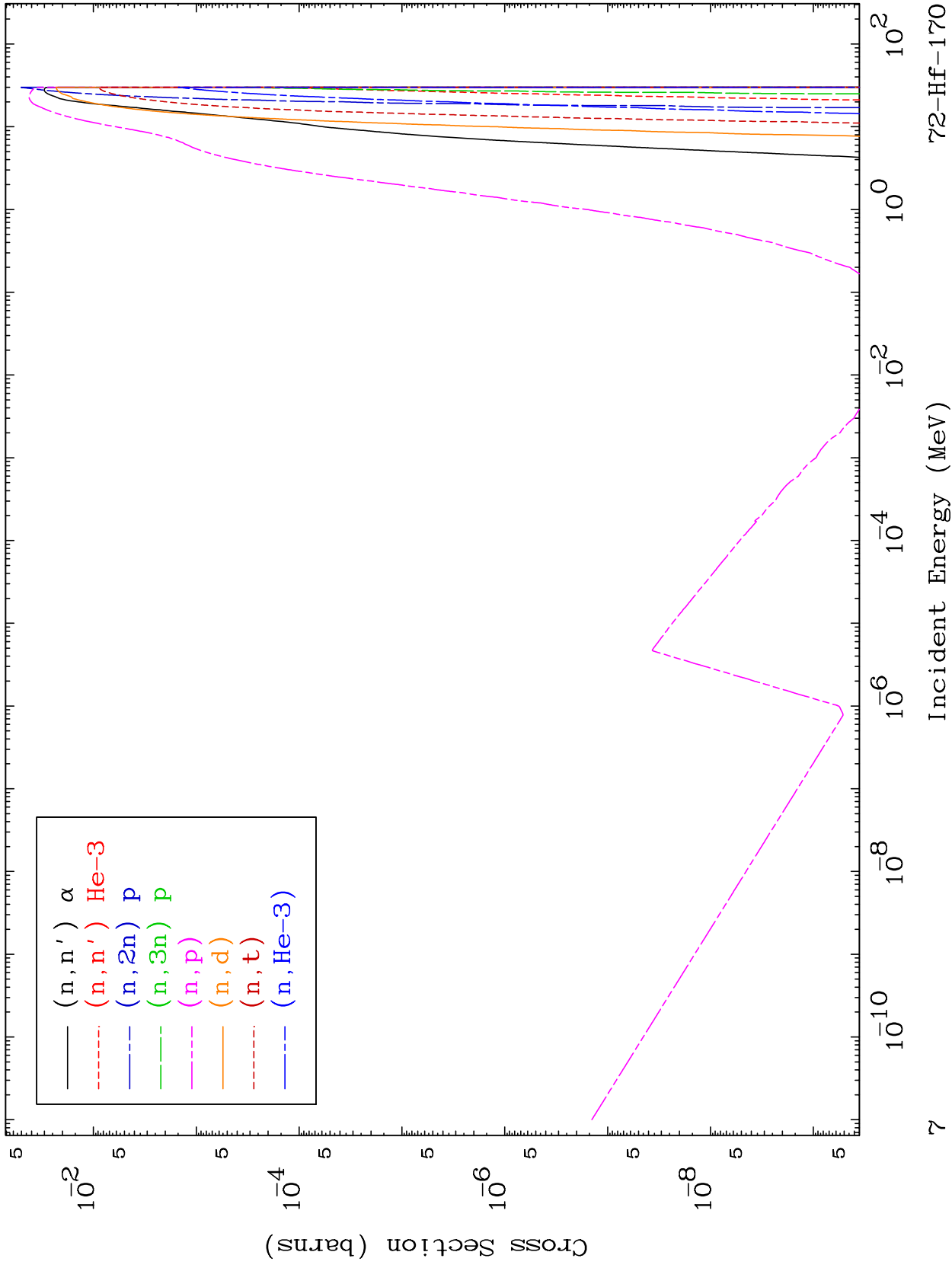
72-Hf-170



MAT 7213

Charged Particle
293 Kelvin Cross Sections

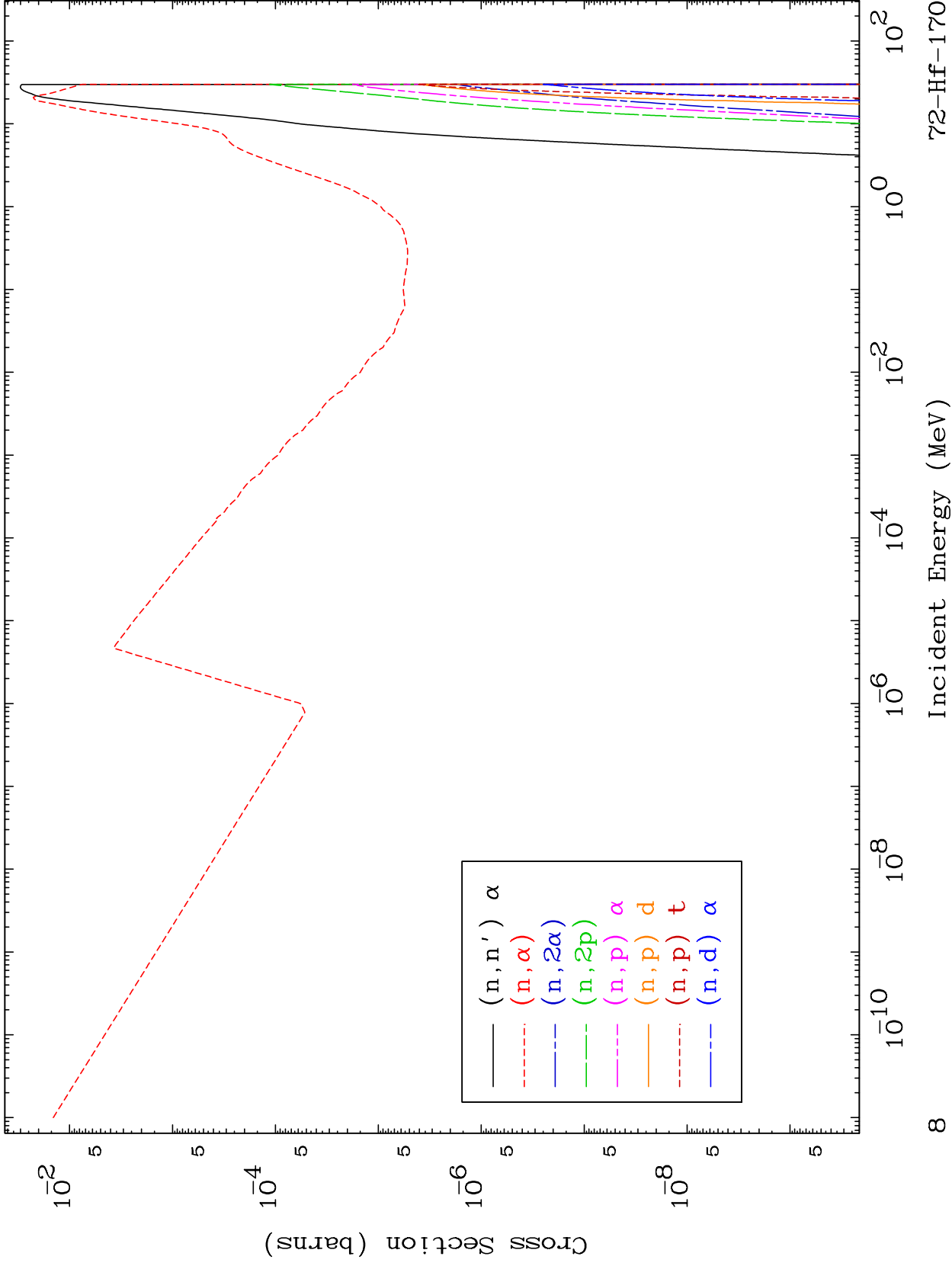
72-Hf-170



MAT 7213

Charged Particle
293 Kelvin Cross Sections

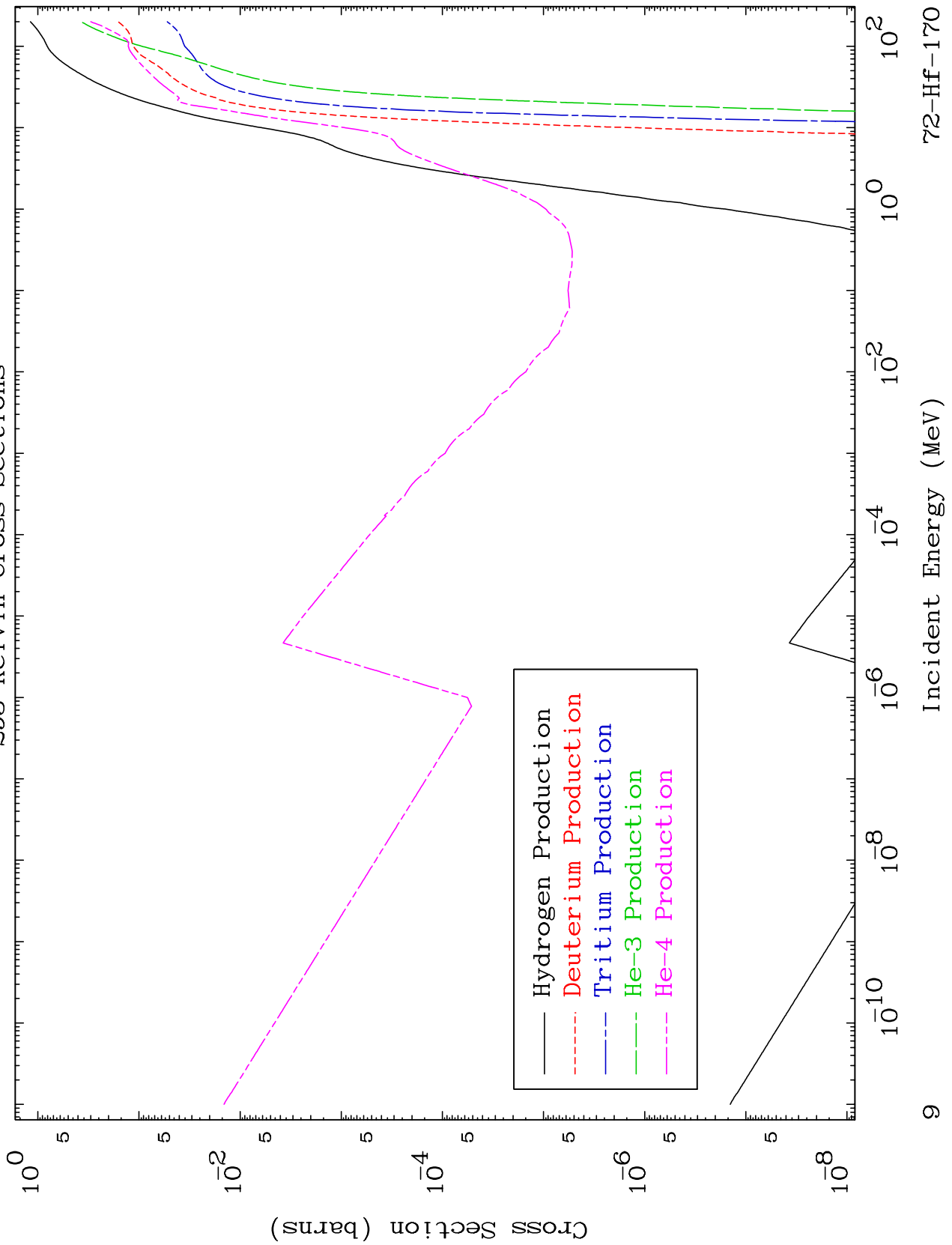
72-Hf-170



MAT 7213

Particle Production
293 Kelvin Cross Sections

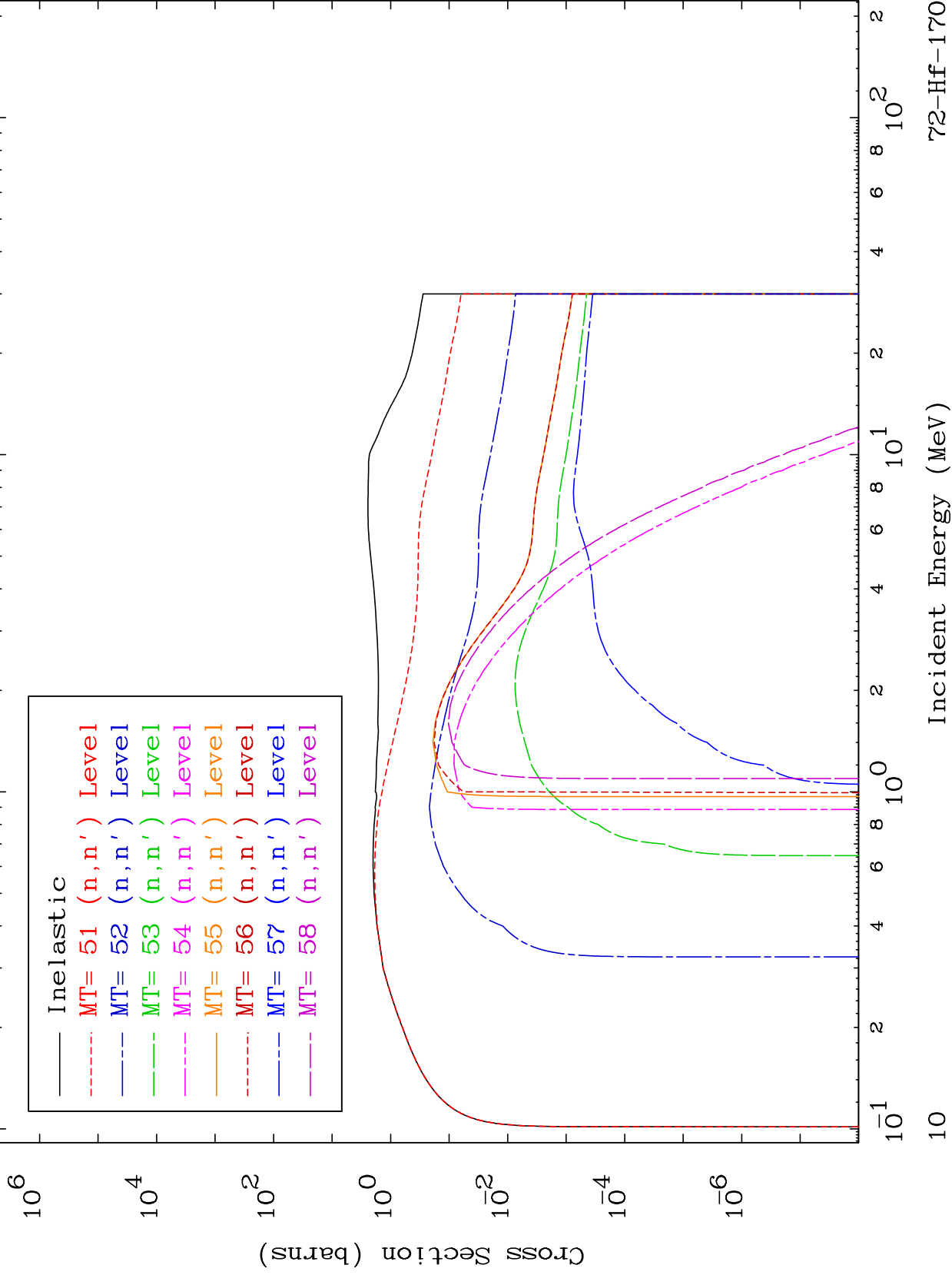
72-Hf-170



MAT 7213

(n,n') Levels
293 Kelvin Cross Sections

72-Hf-170



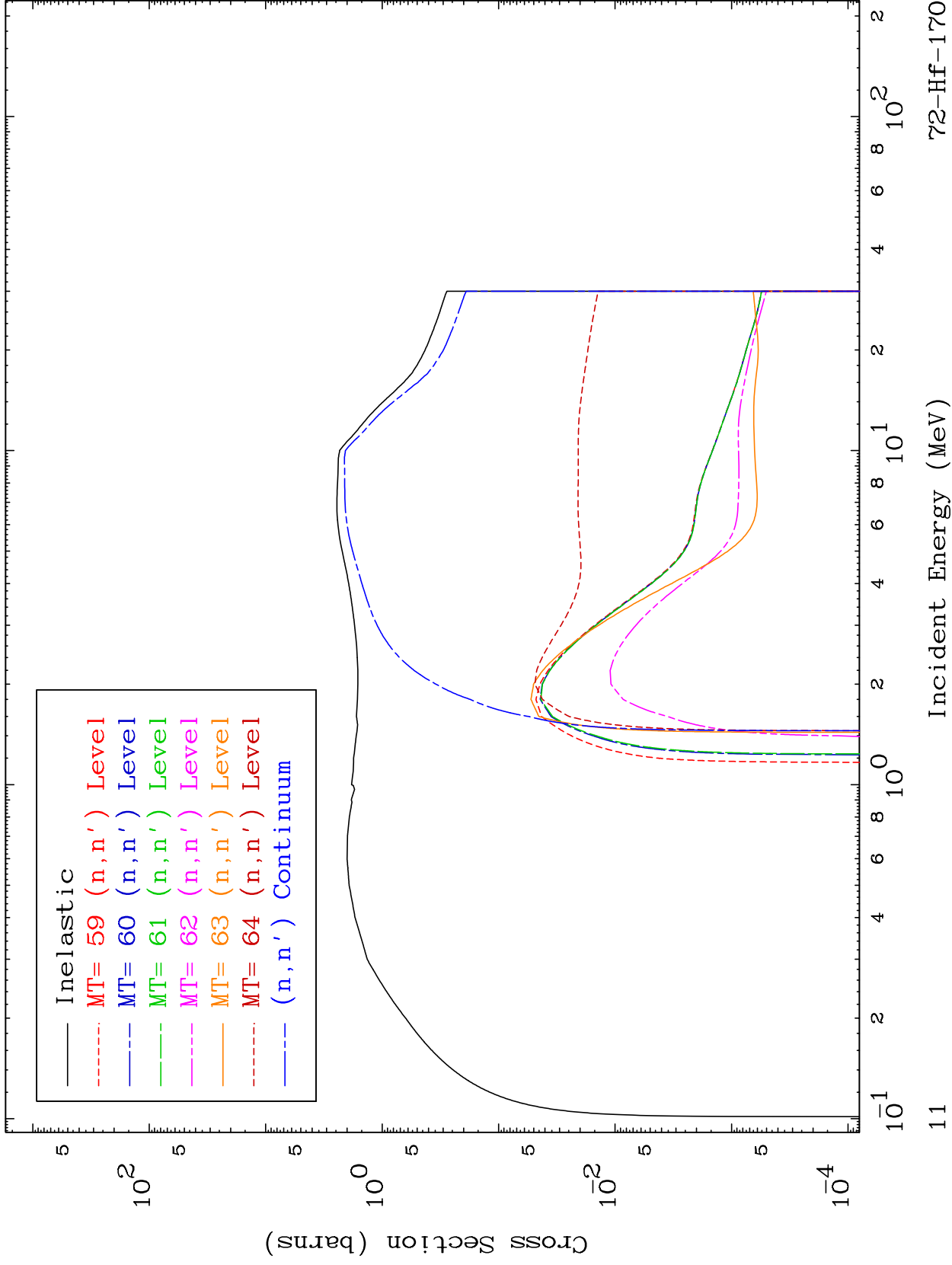
72-Hf-170

Incident Energy (MeV)

MAT 7213

(n,n') Levels
293 Kelvin Cross Sections

72-Hf-170



11

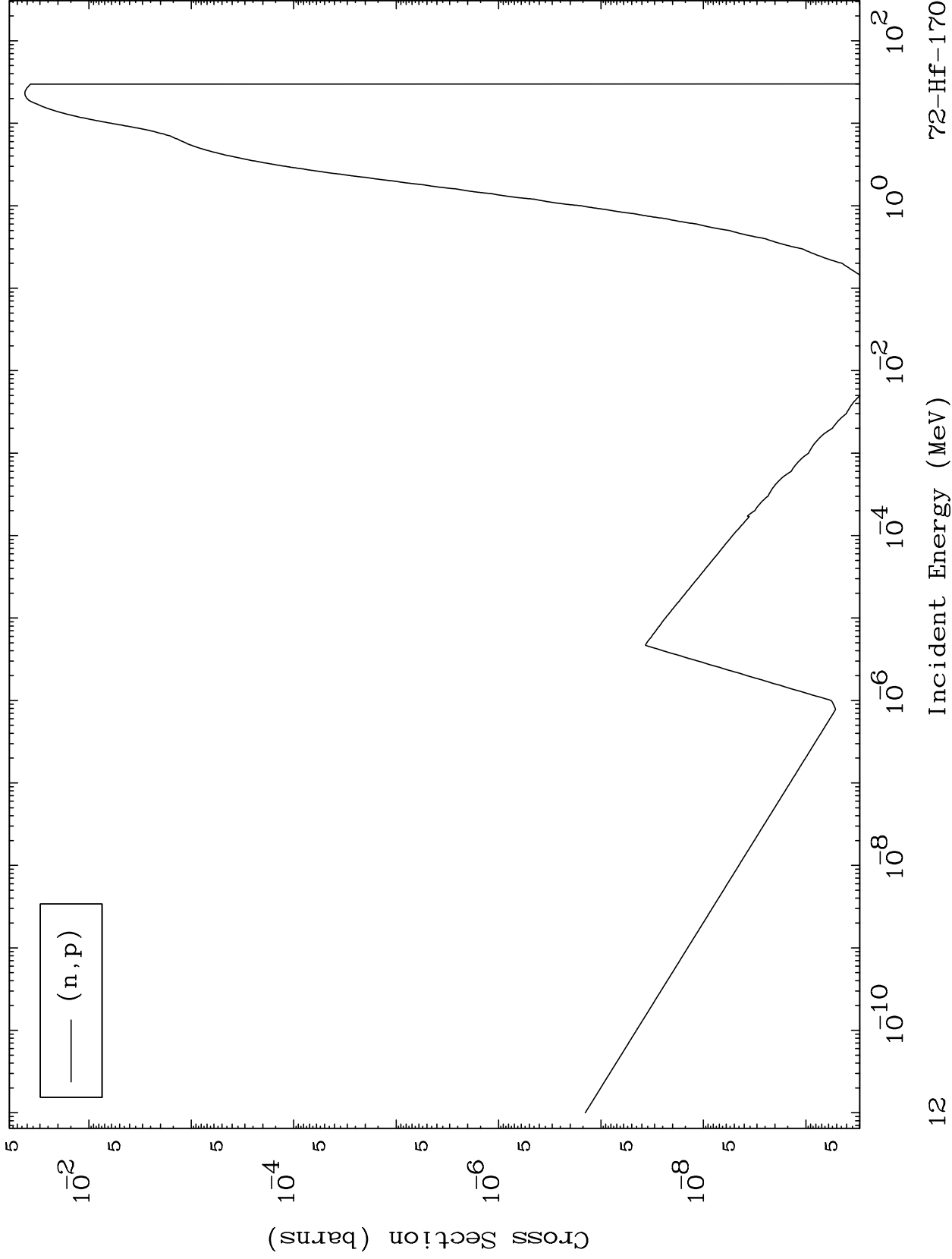
Incident Energy (MeV)

72-Hf-170

MAT 7213

(n,p) Levels
293 Kelvin Cross Sections

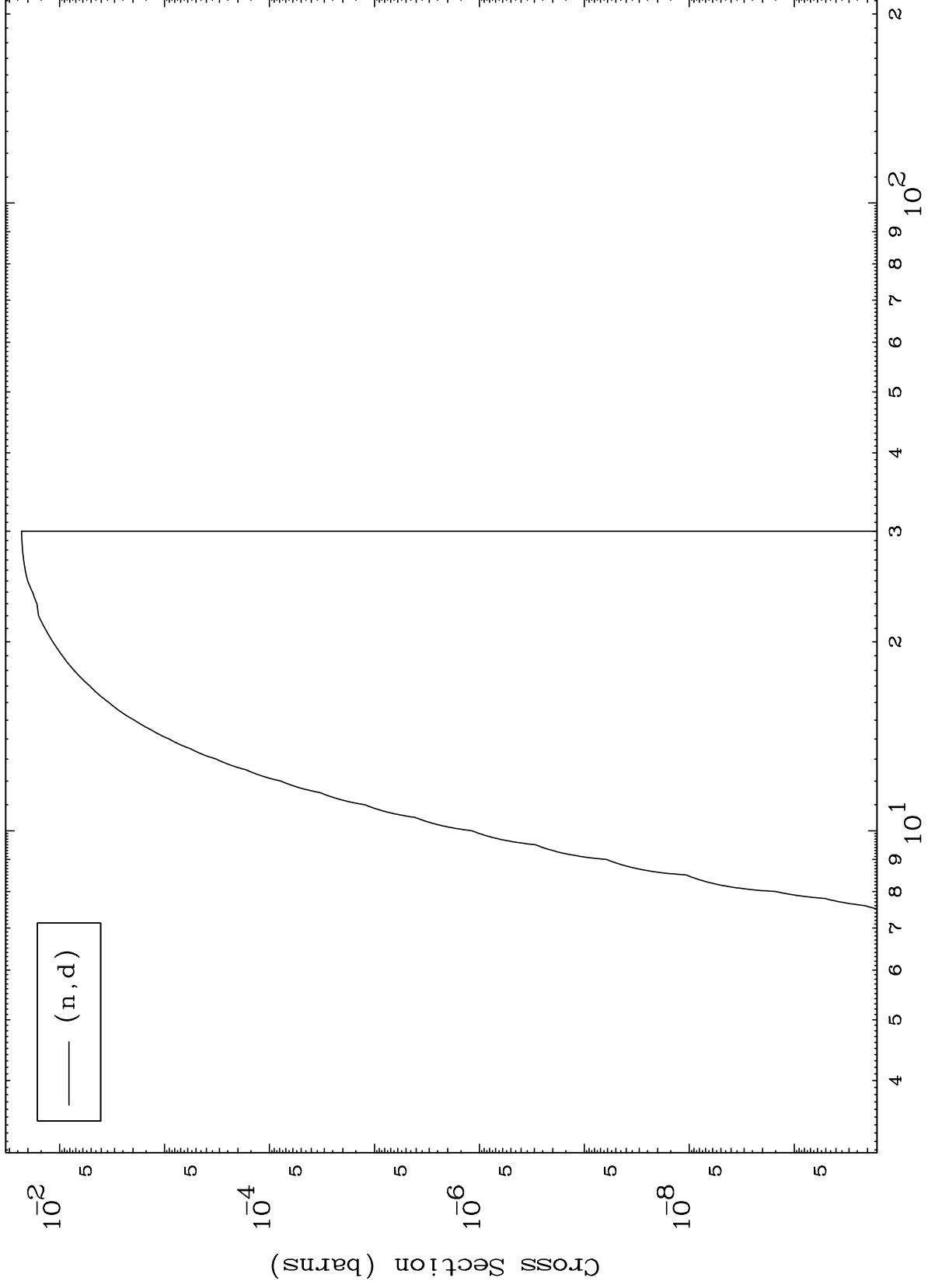
72-Hf-170



MAT 7213

(n,d) Levels
293 Kelvin Cross Sections

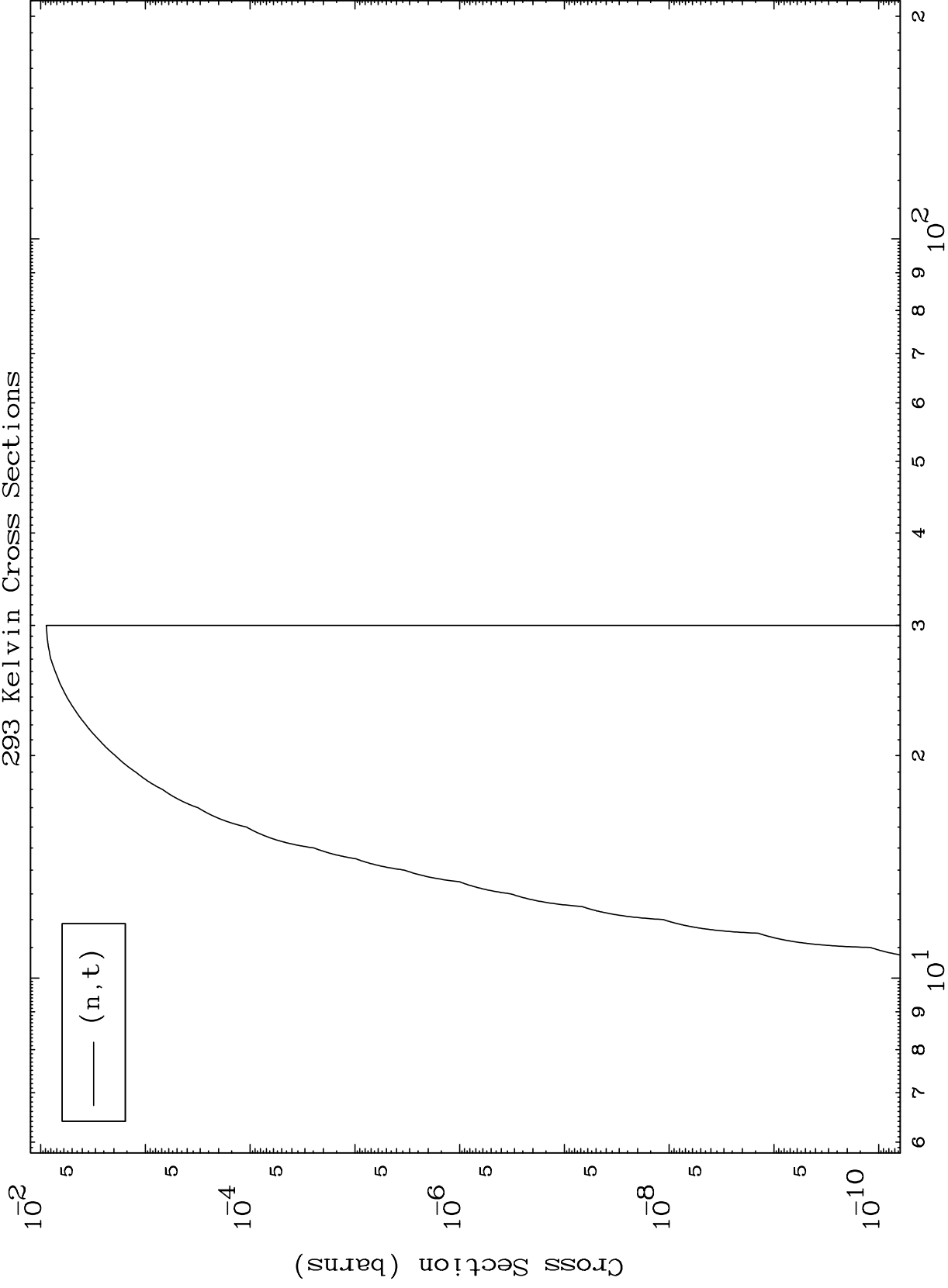
72-Hf-170



MAT 7213

(n,t) Levels
293 Kelvin Cross Sections

72-Hf-170



14

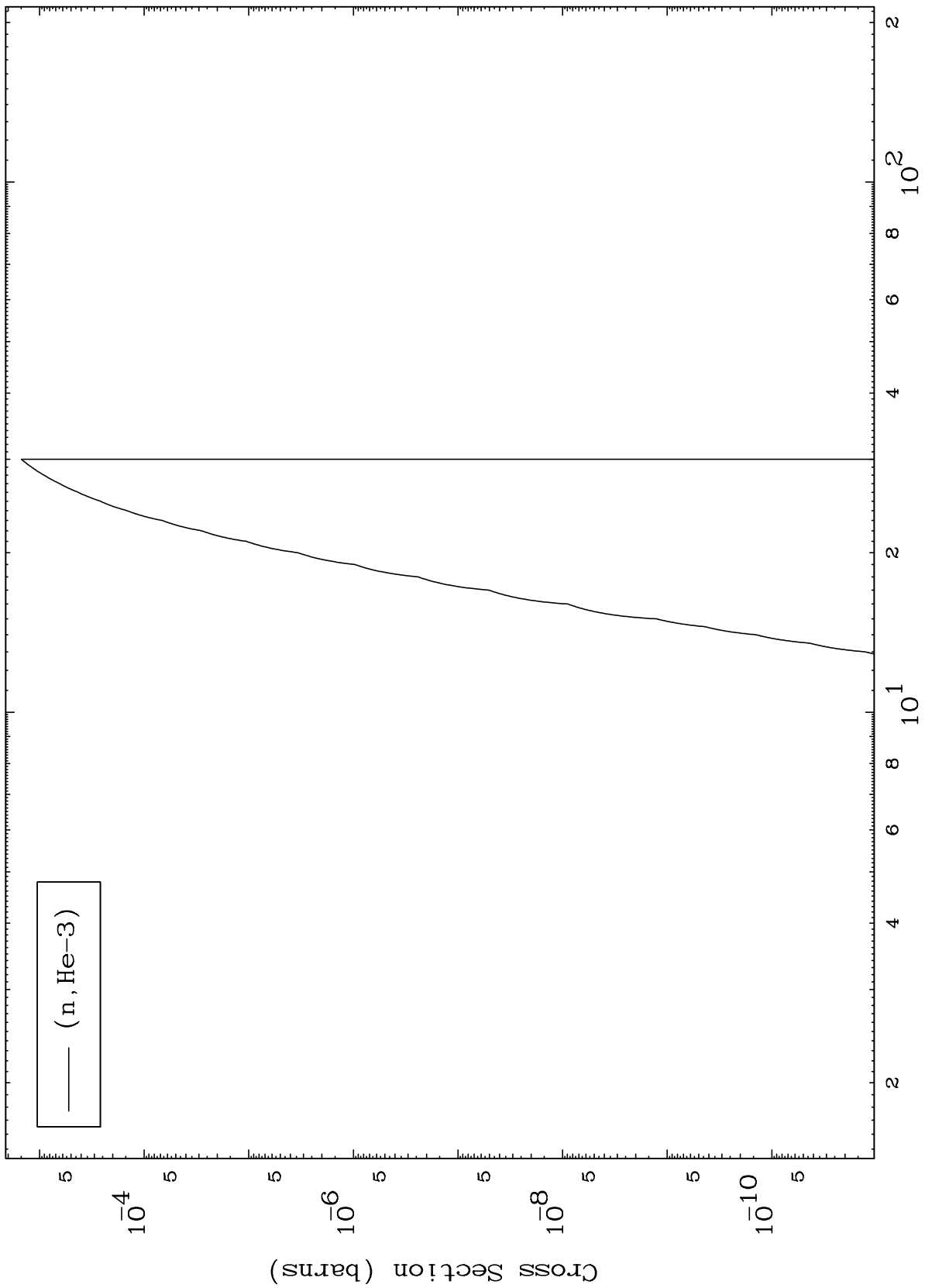
Incident Energy (MeV)

72-Hf-170

MAT 7213

(n,He3) Levels
293 Kelvin Cross Sections

72-Hf-170



15

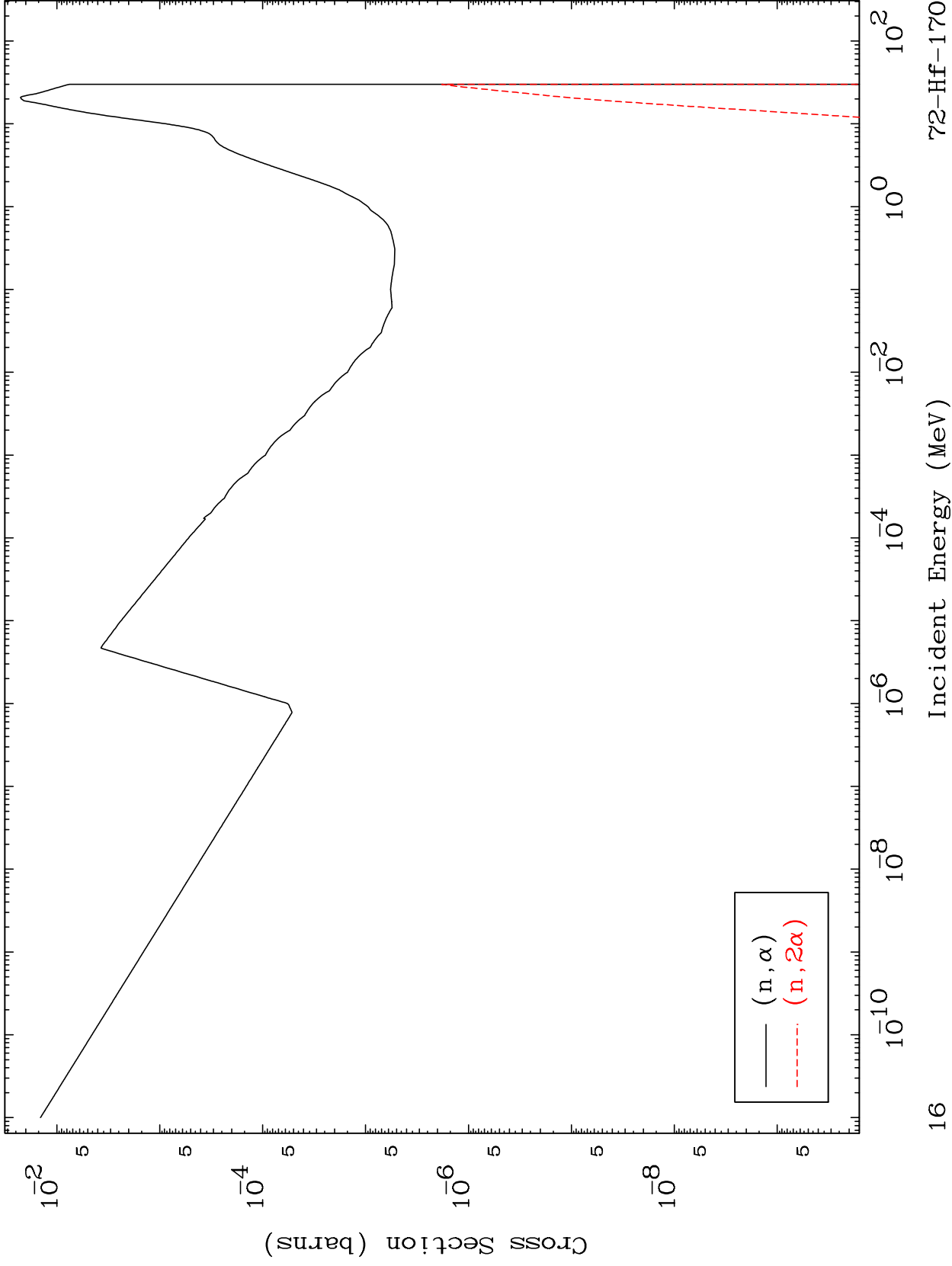
Incident Energy (MeV)

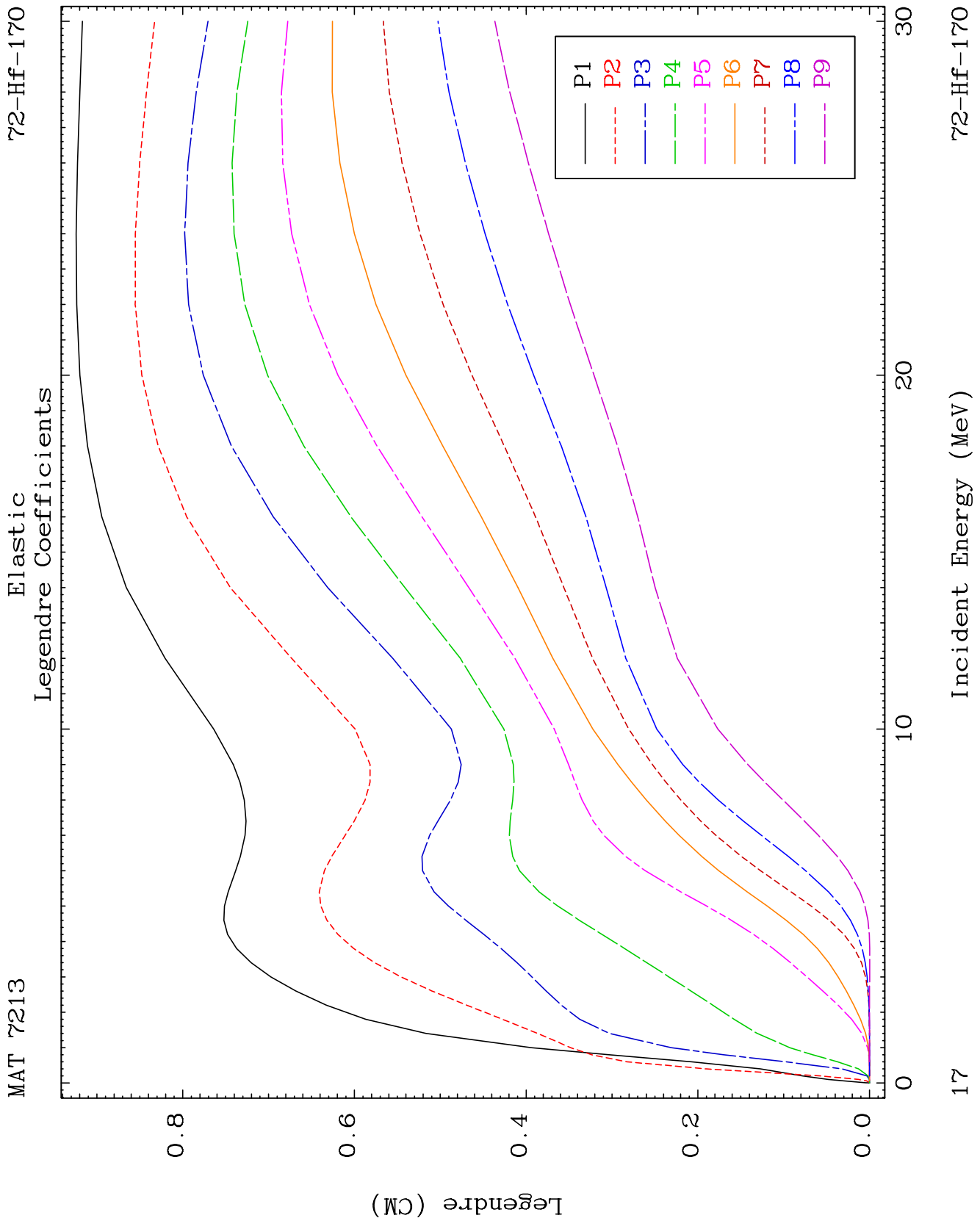
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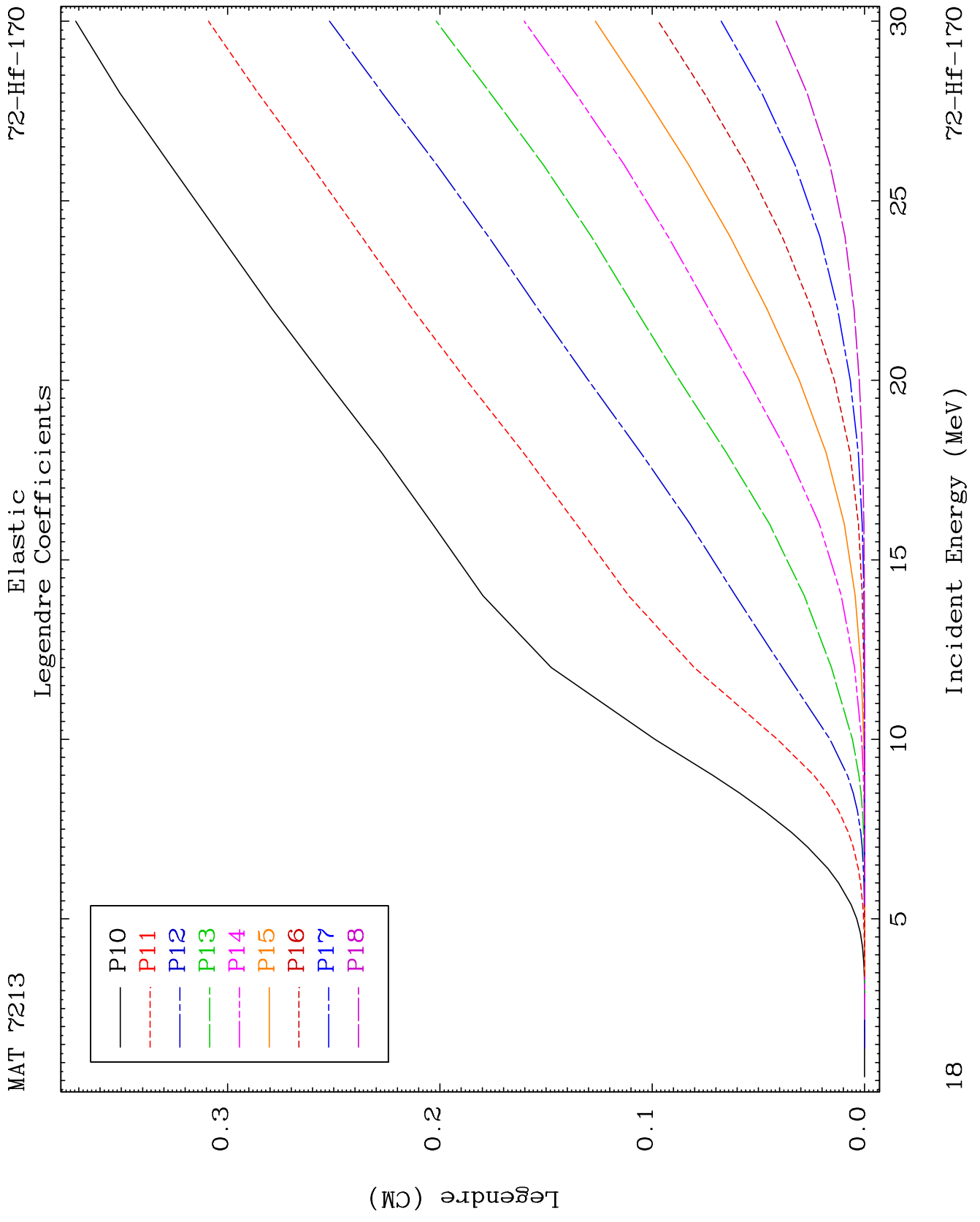
MAT 7213

(n,α) Levels
293 Kelvin Cross Sections

72-Hf-170



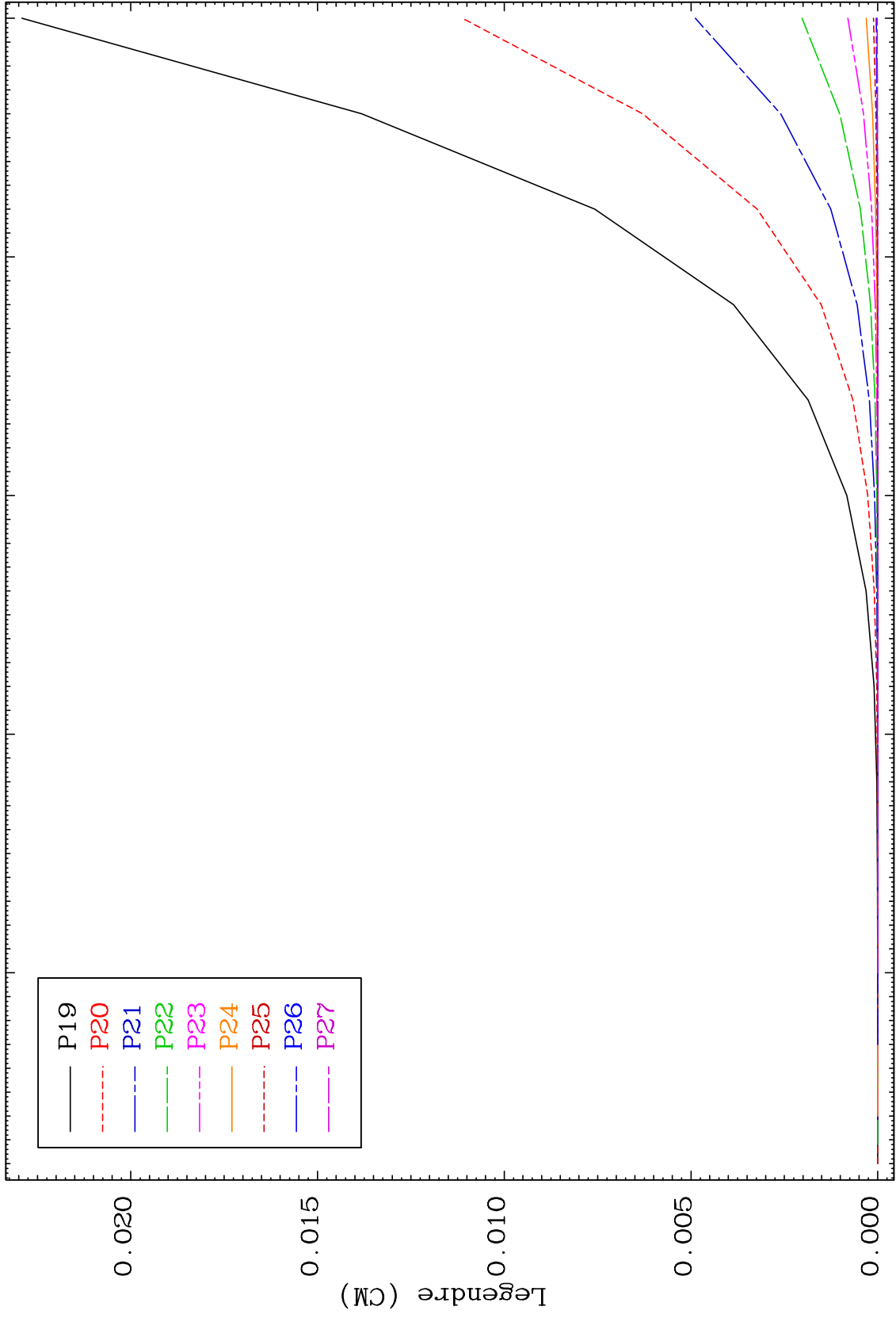




MAT 7213

Elastic Legendre Coefficients

72-Hf-170



19

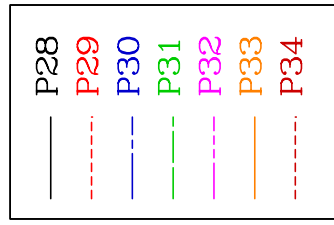
Incident Energy (MeV)

72-Hf-170

MAT 7213

Elastic
Legendre Coefficients

72-Hf-170

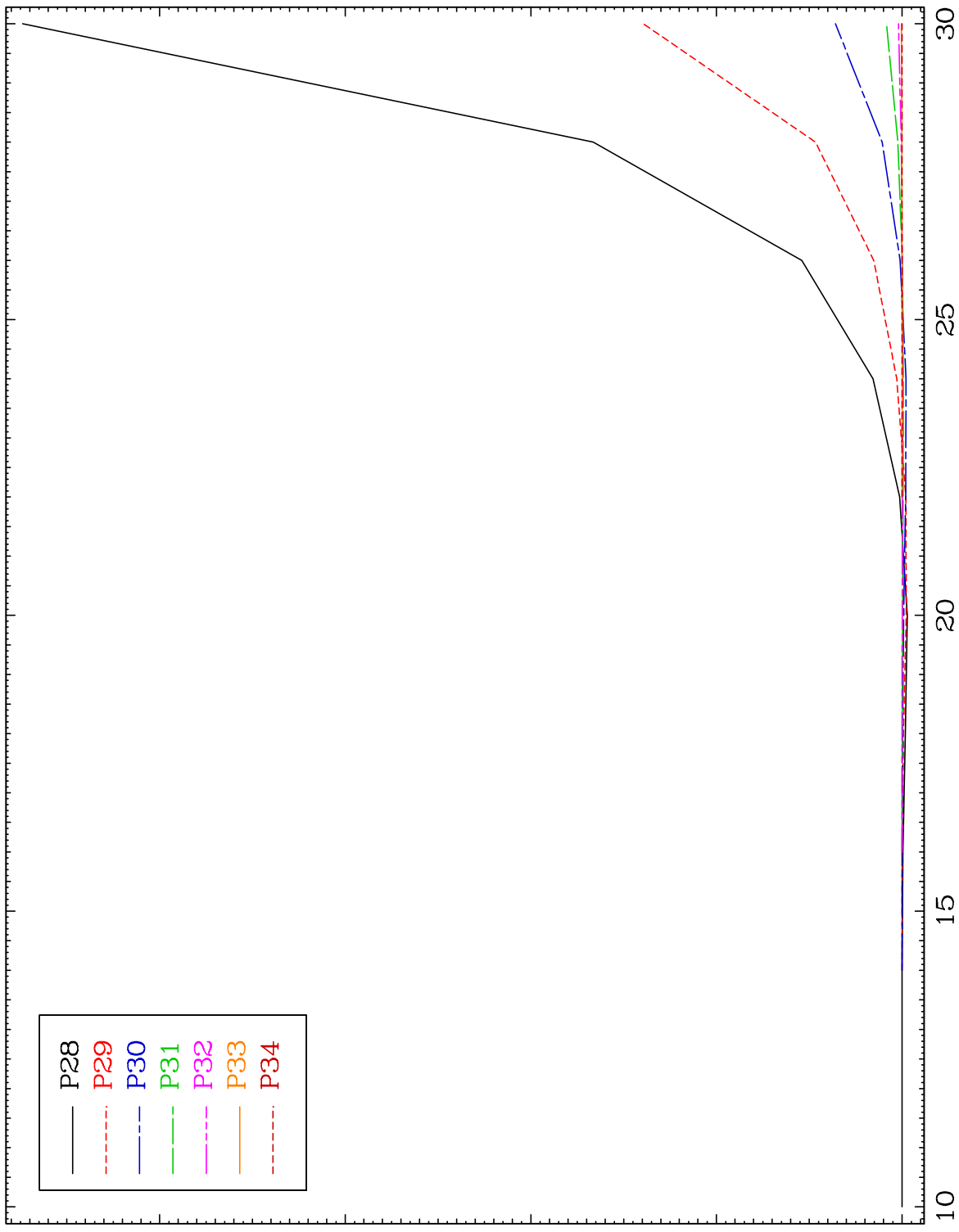


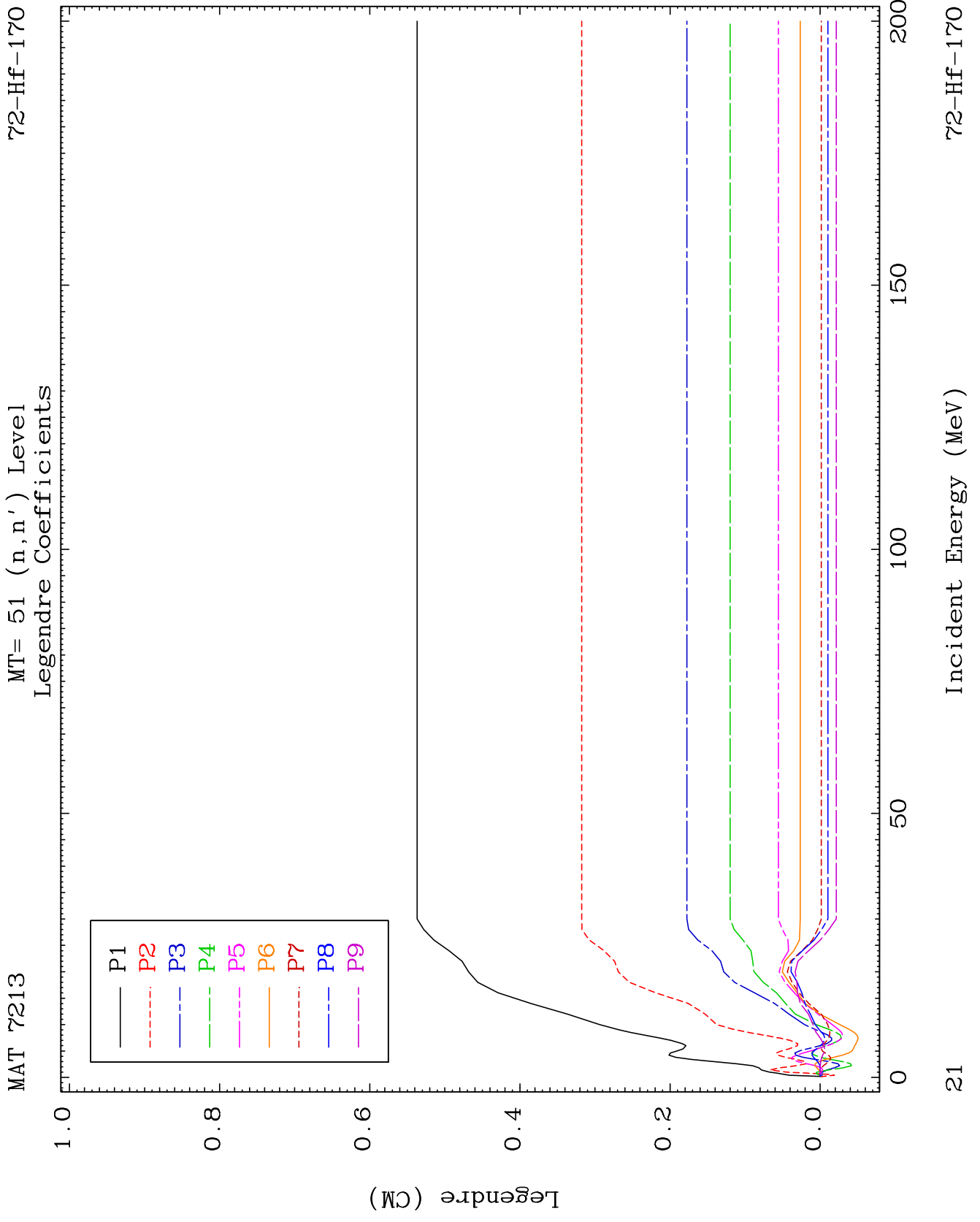
$\times 10^{-6}$

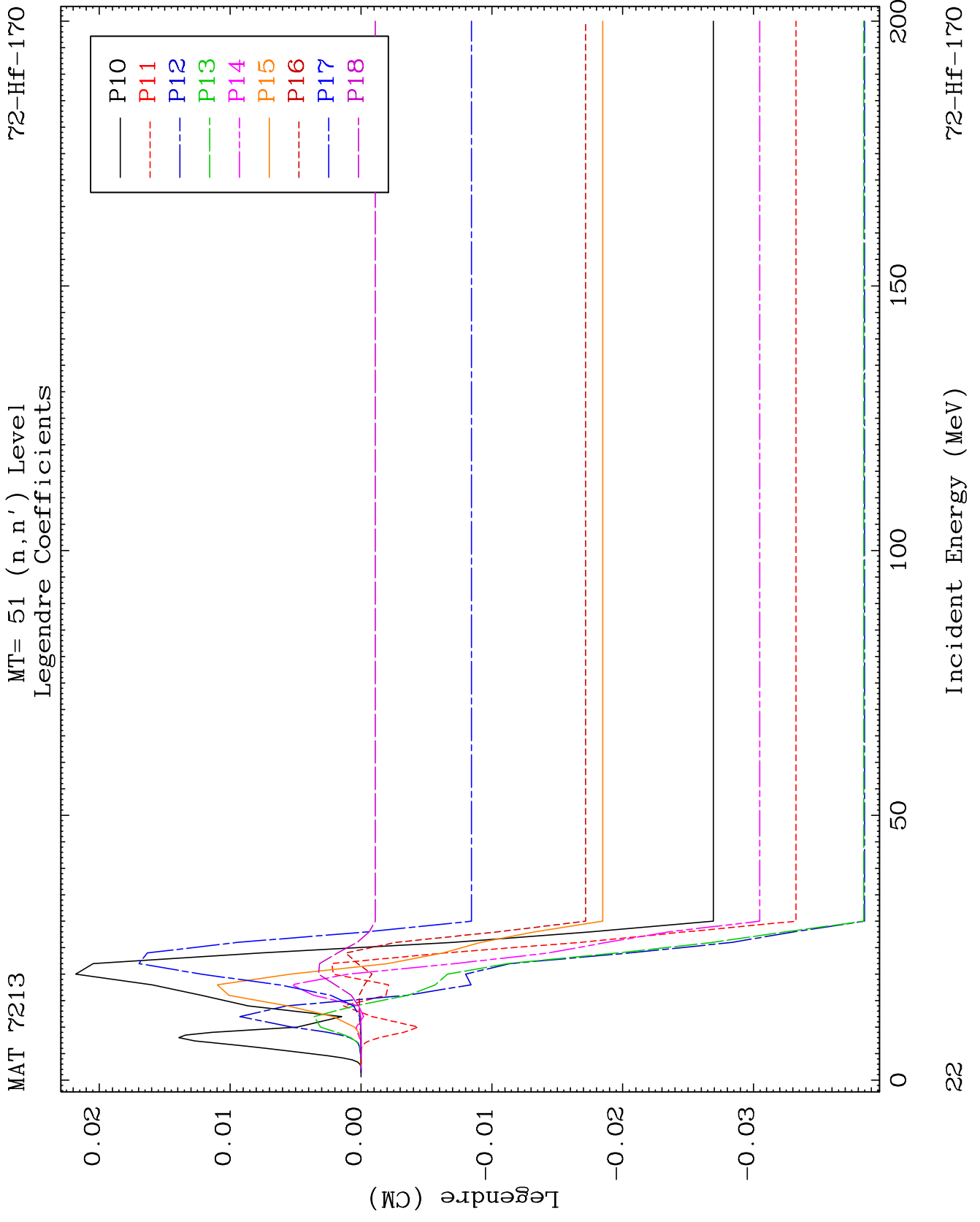
Legendre (CM)

Incident Energy (MeV)

72-Hf-170



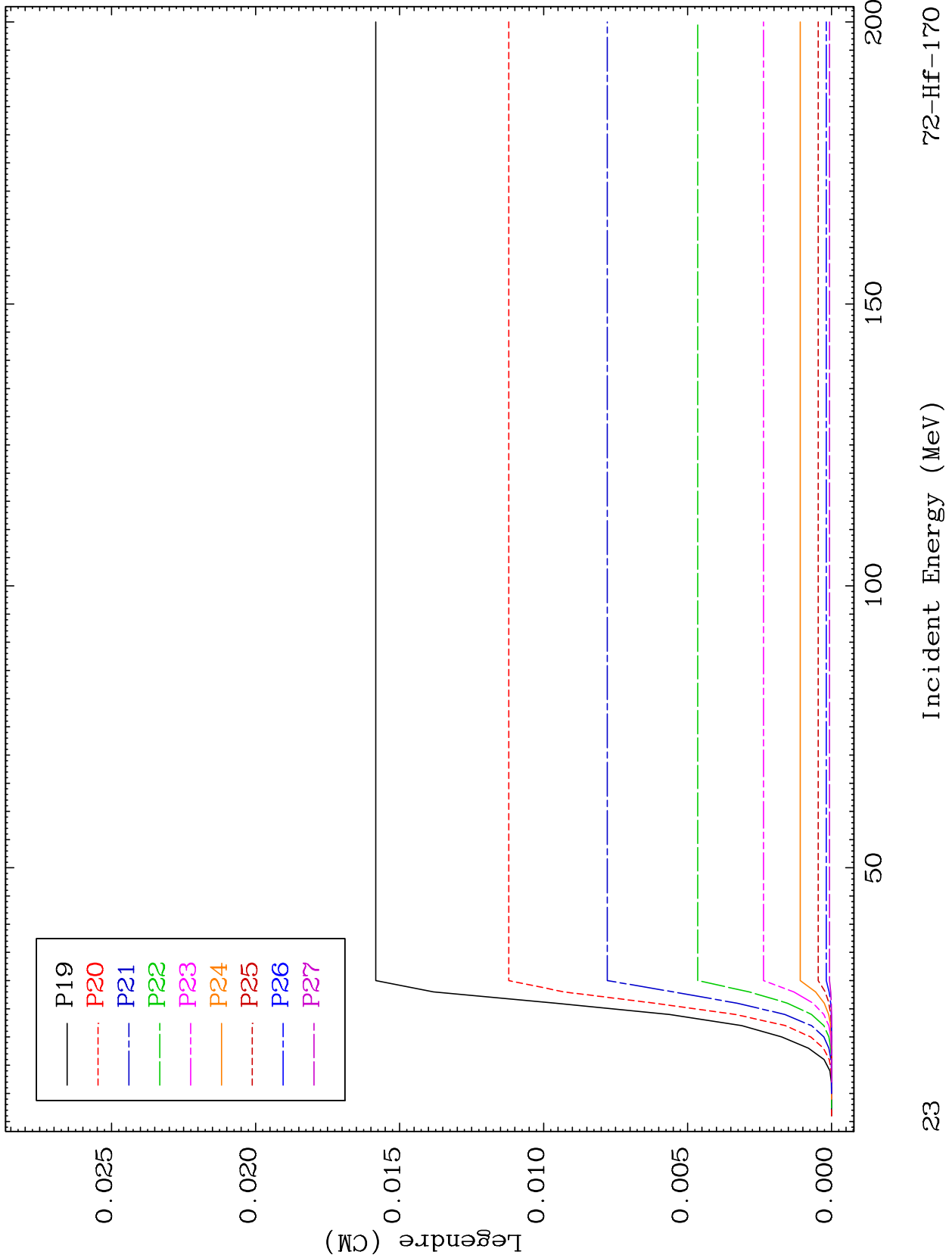




MAT 7213

MT= 51 (n,n') Level
Legendre Coefficients

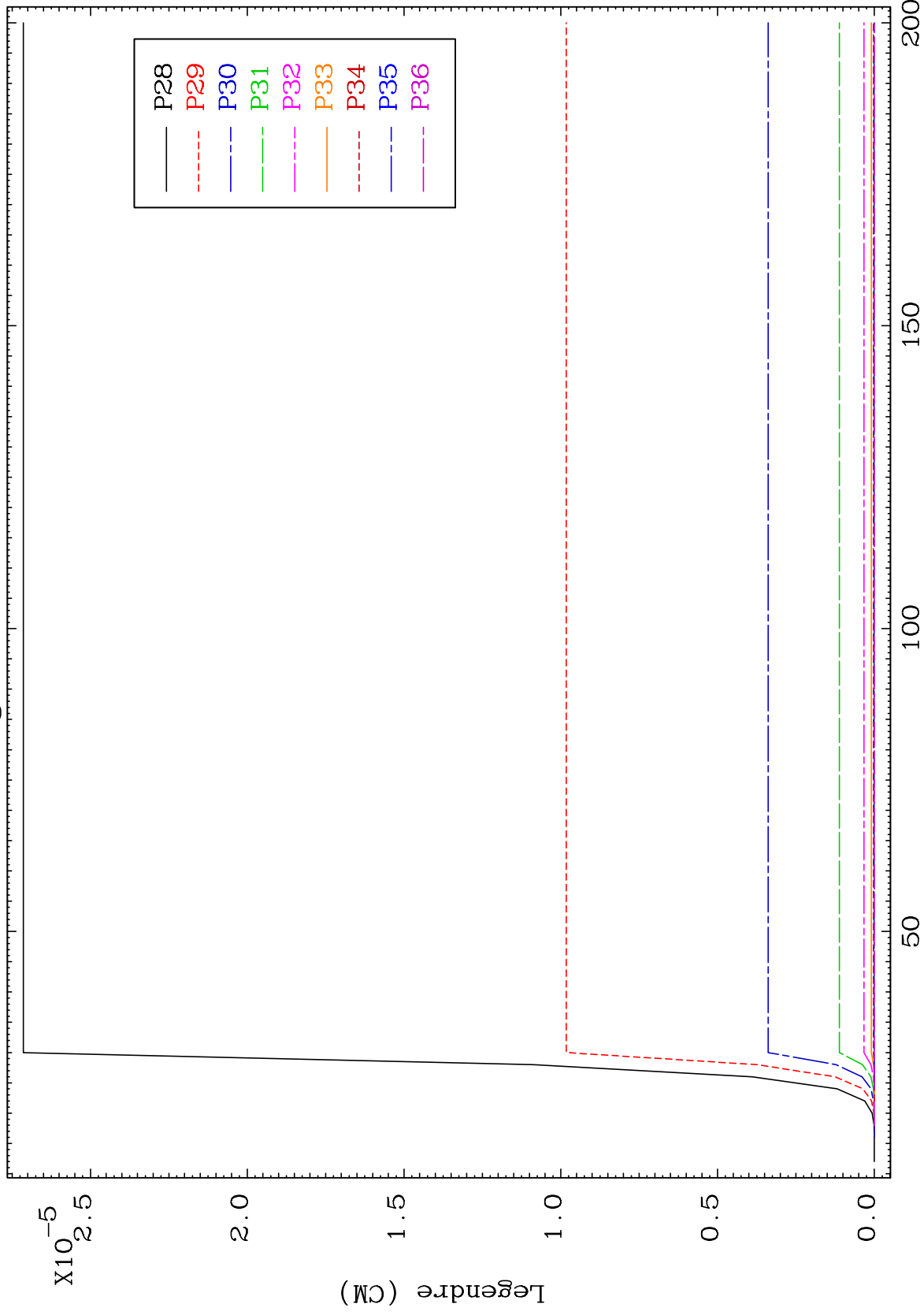
72-Hf-170



MAT 7213

MT= 51 (n,n') Level
Legendre Coefficients

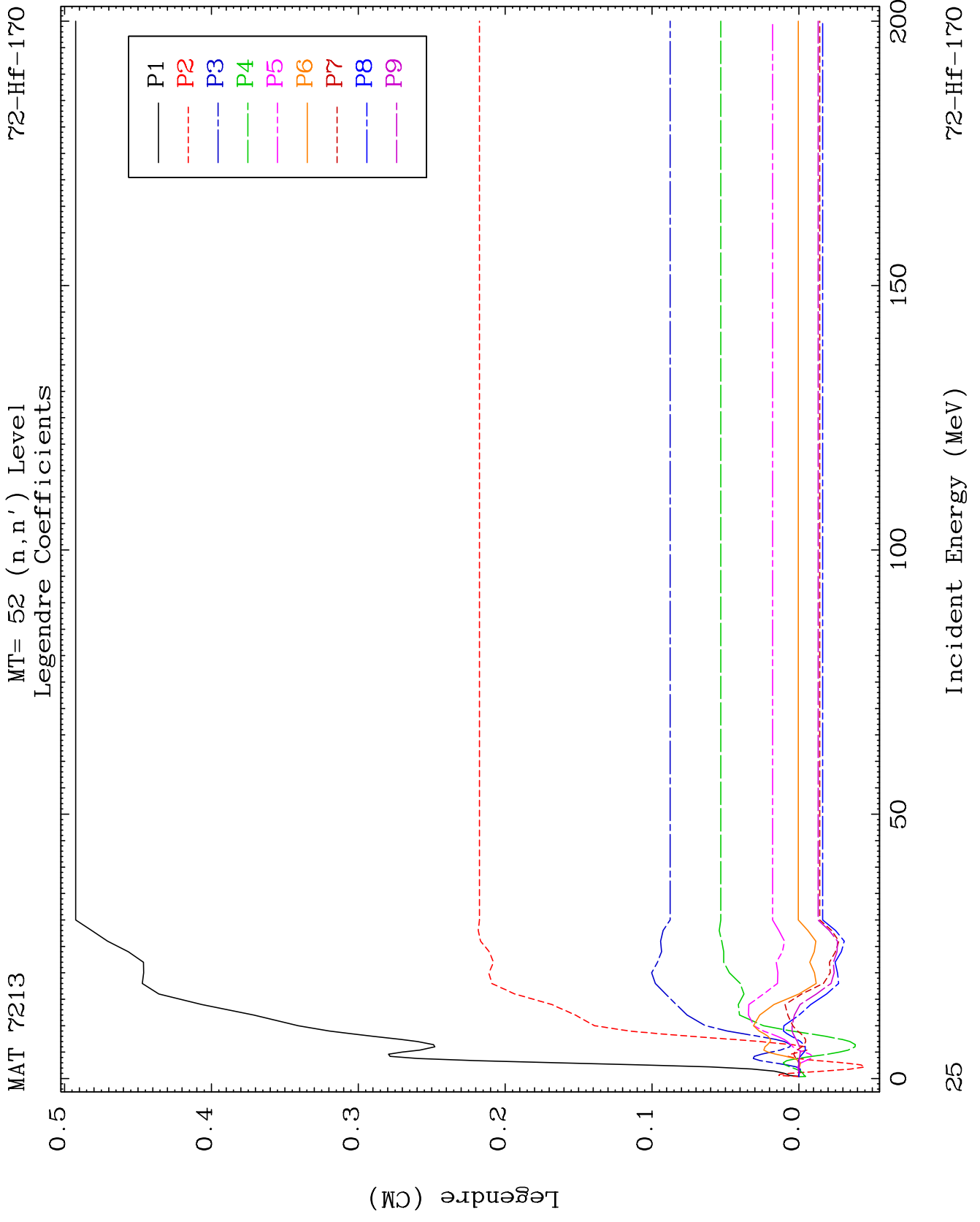
72-Hf-170



24

Incident Energy (MeV)

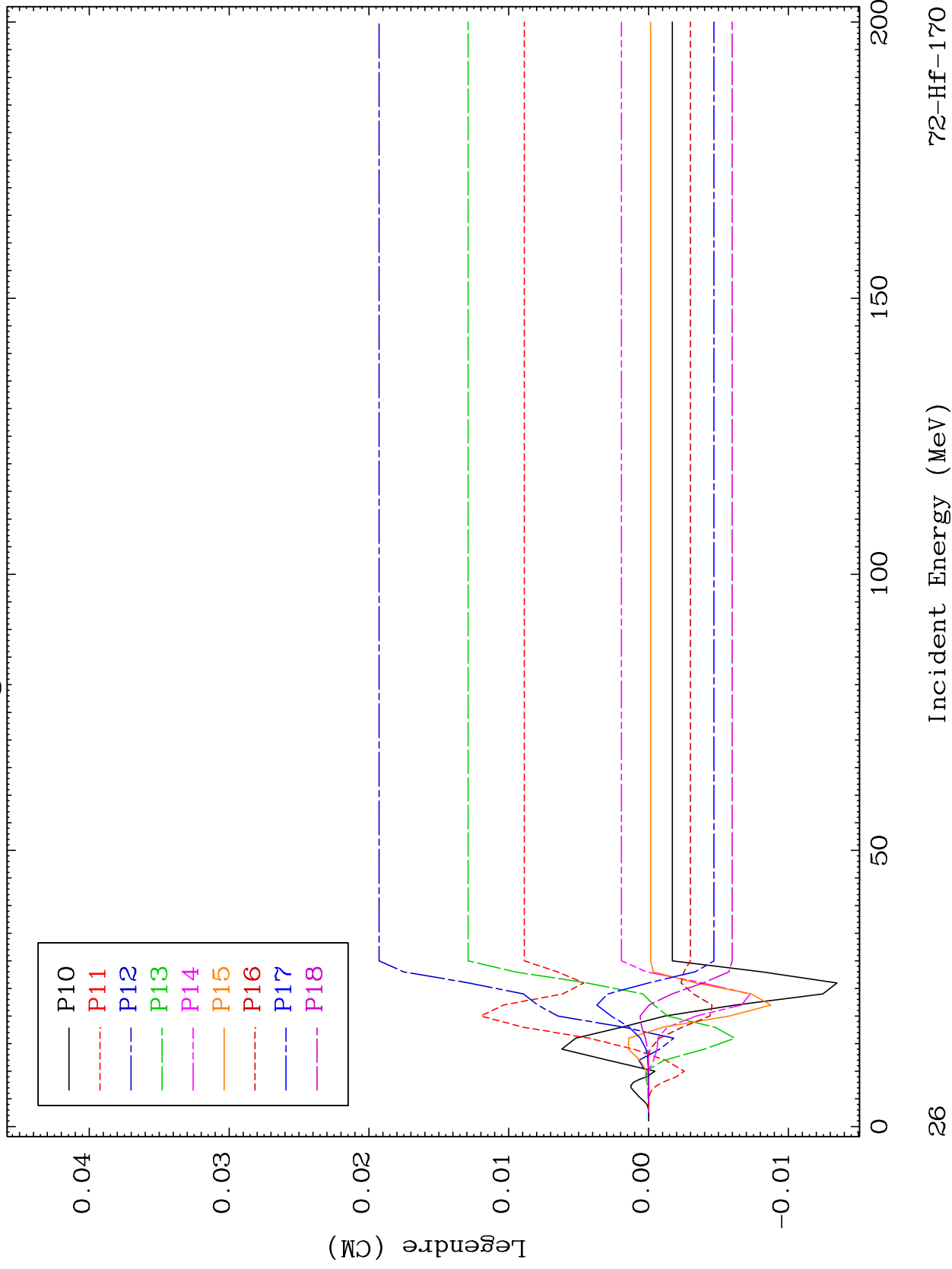
72-Hf-170



MAT 7213

MT= 52 (n,n') Level
Legendre Coefficients

72-Hf-170



26

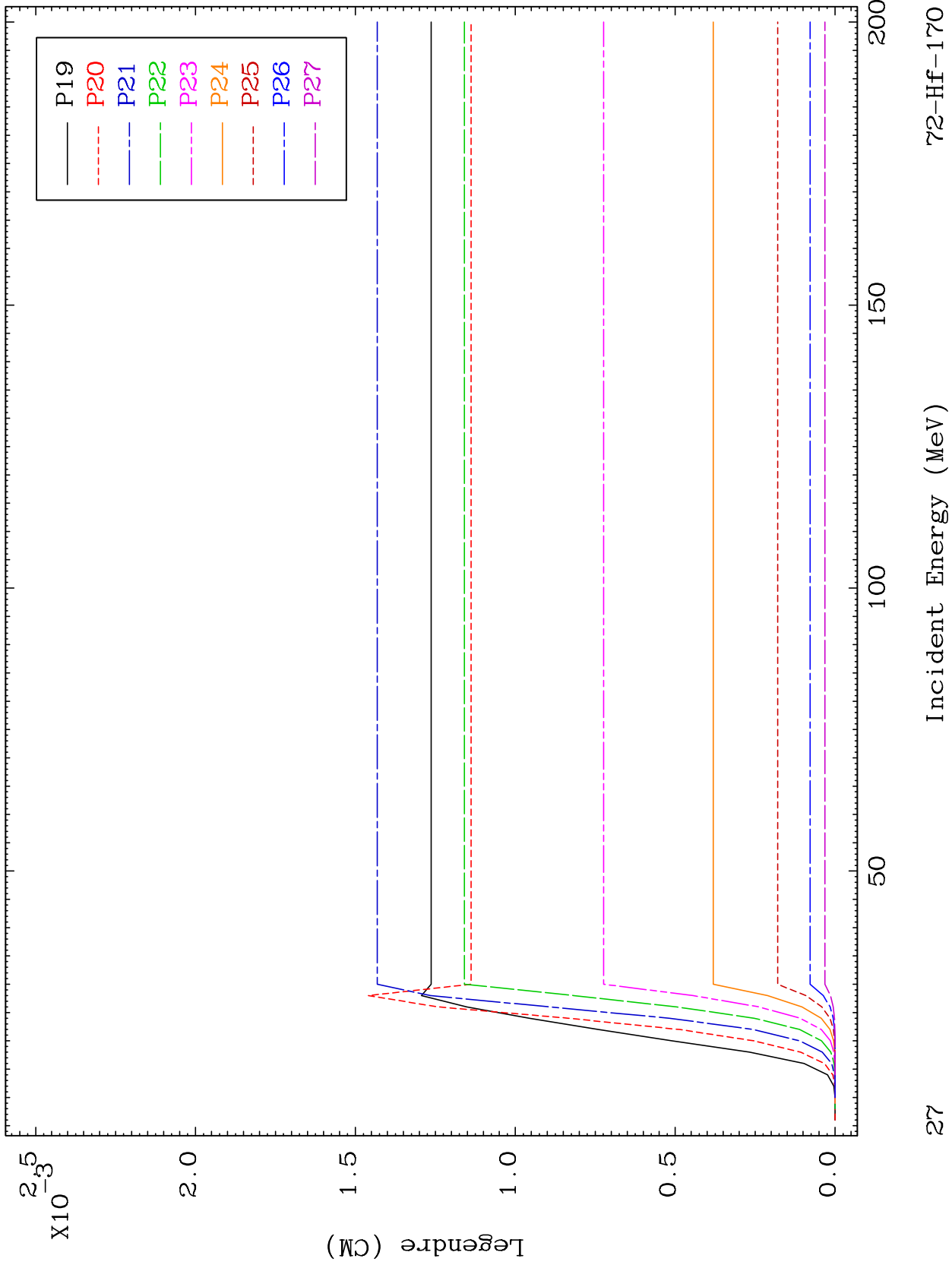
Incident Energy (MeV)

72-Hf-170

MAT 7213

MT= 52 (n,n') Level
Legendre Coefficients

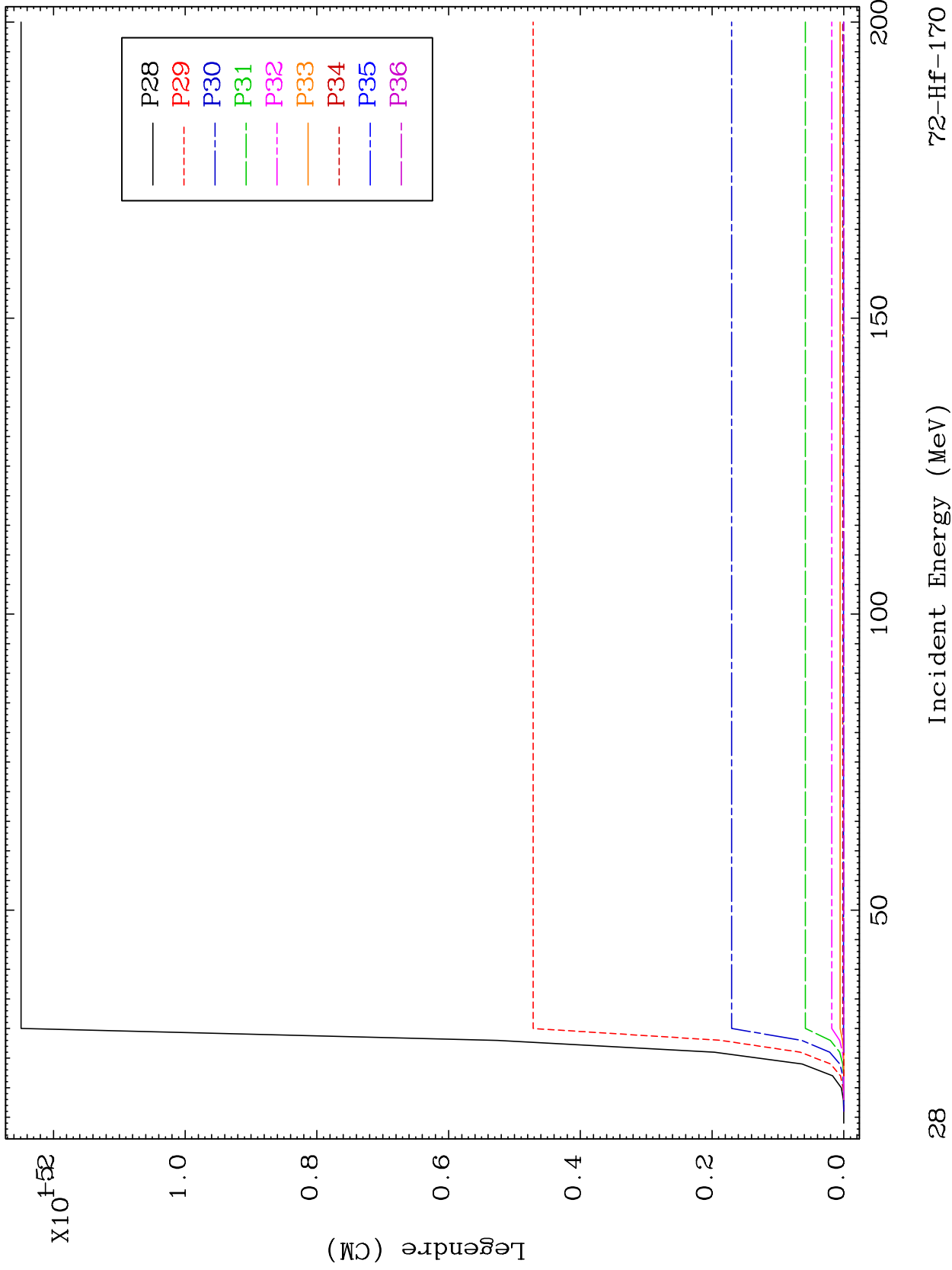
72-Hf-170



MAT 7213

MT= 52 (n,n') Level
Legendre Coefficients

72-Hf-170



28

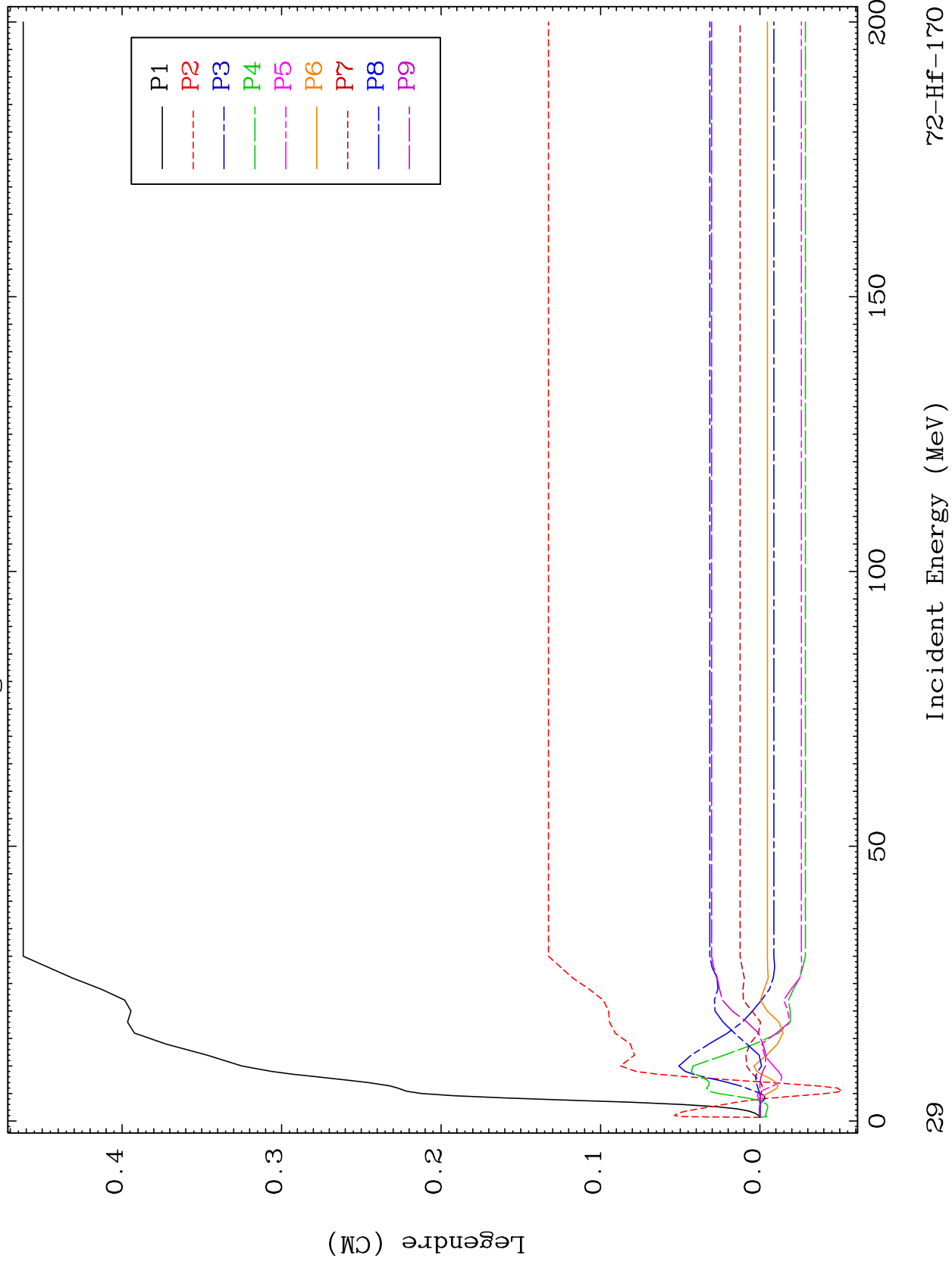
Incident Energy (MeV)

72-Hf-170

MAT 7213

MT= 53 (n,n') Level
Legendre Coefficients

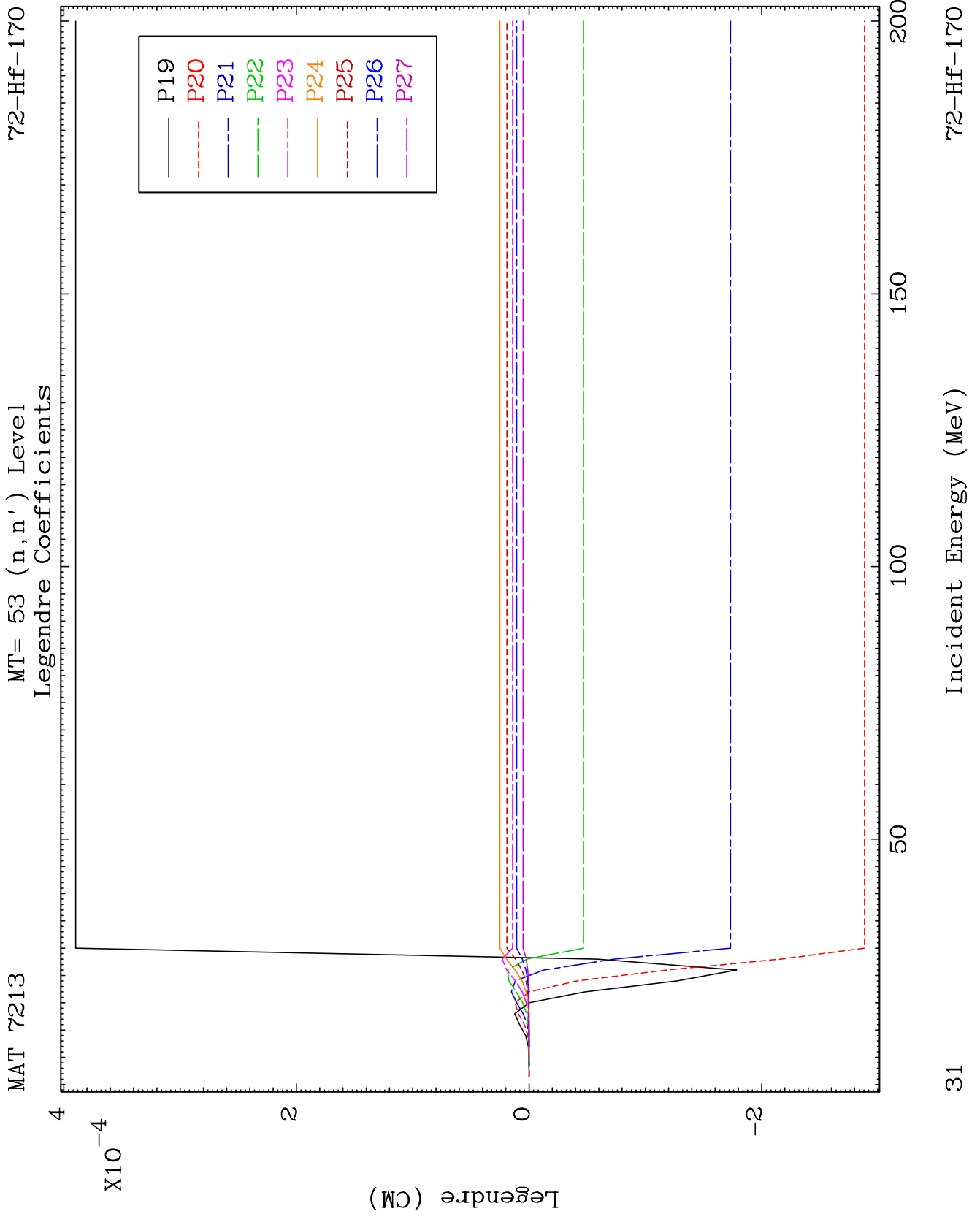
72-Hf-170



29

Incident Energy (MeV)

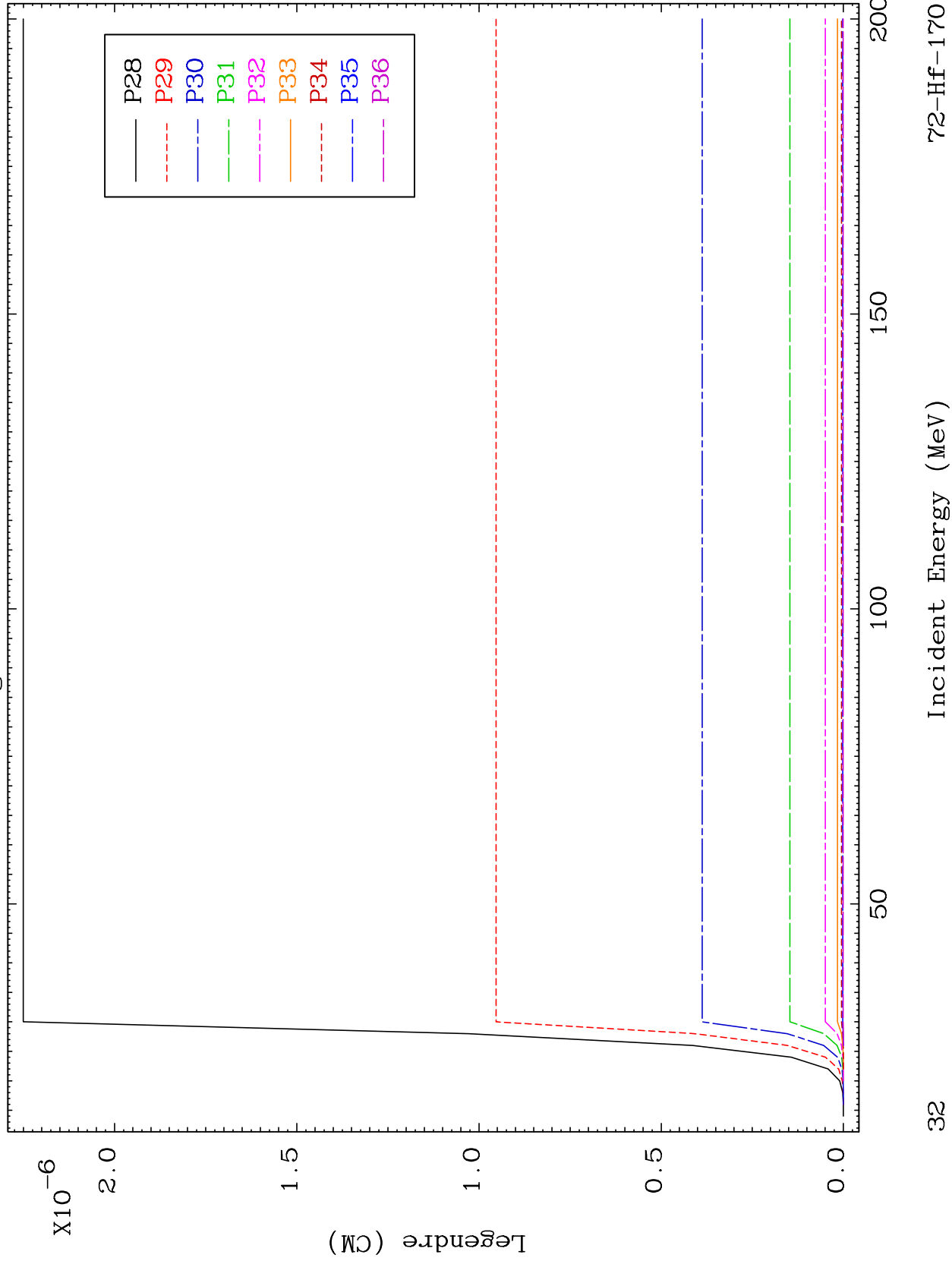
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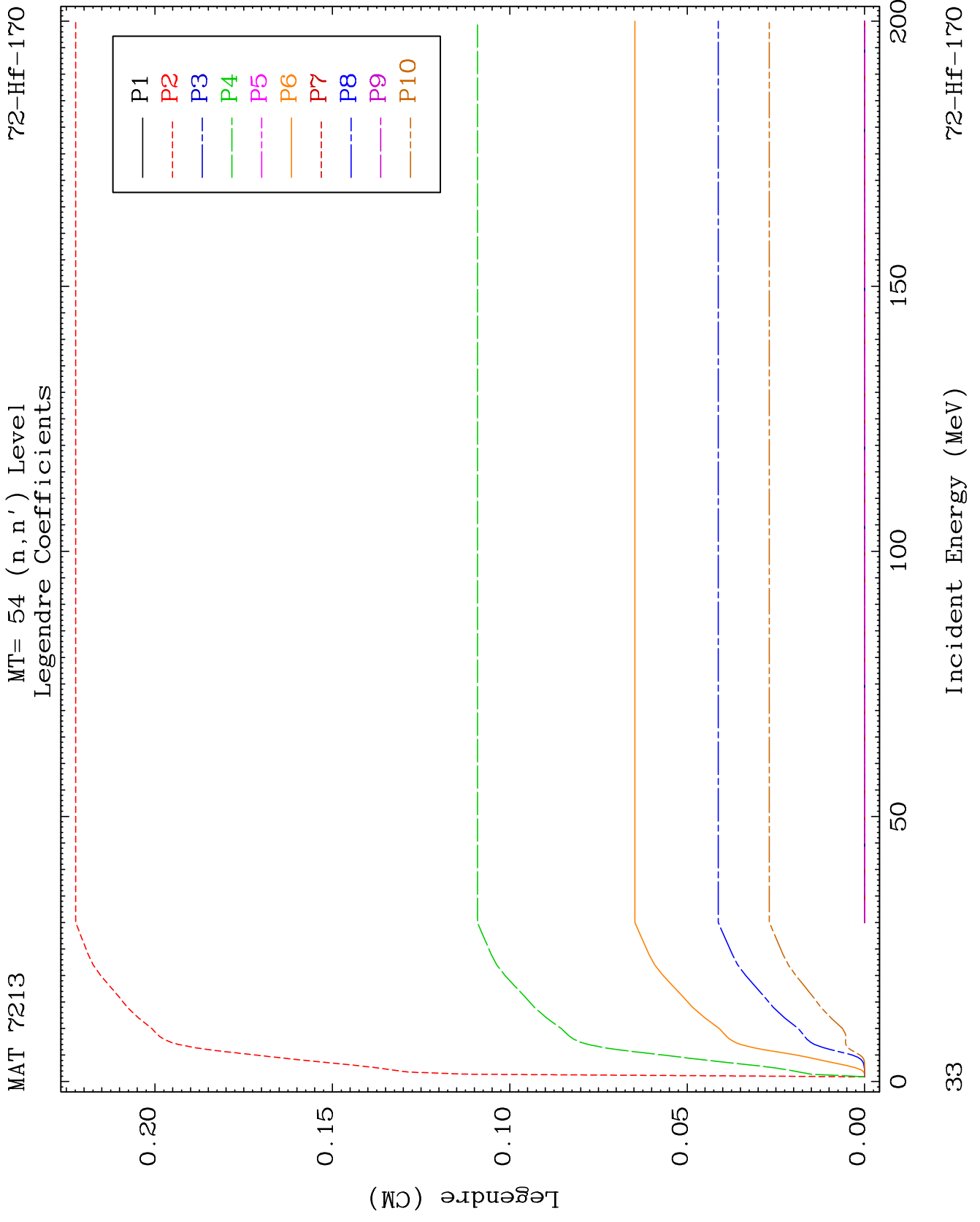
MAT 7213

MT= 53 (n,n') Level
Legendre Coefficients

72-Hf-170



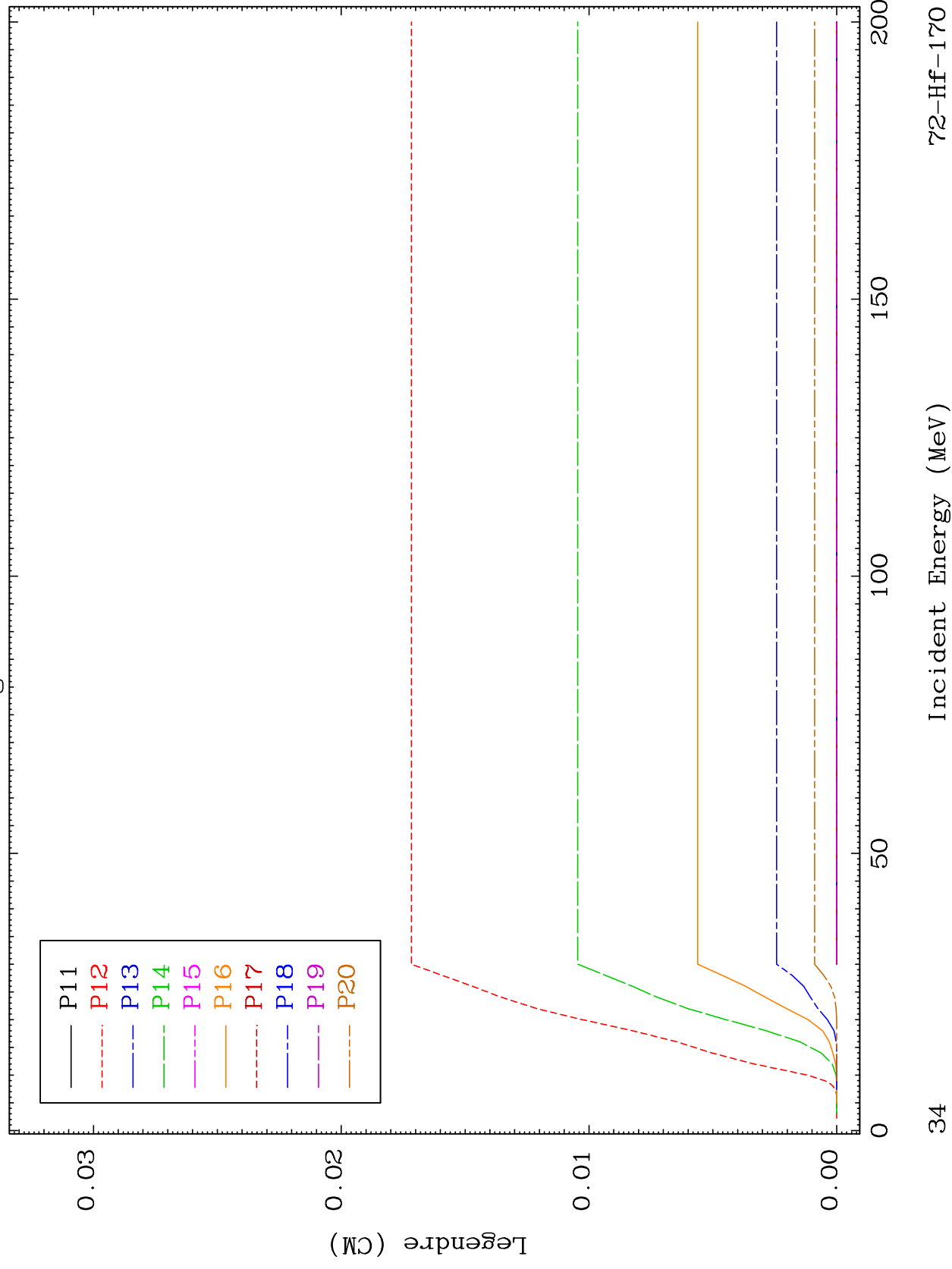
32



MAT 7213

MT= 54 (n,n') Level
Legendre Coefficients

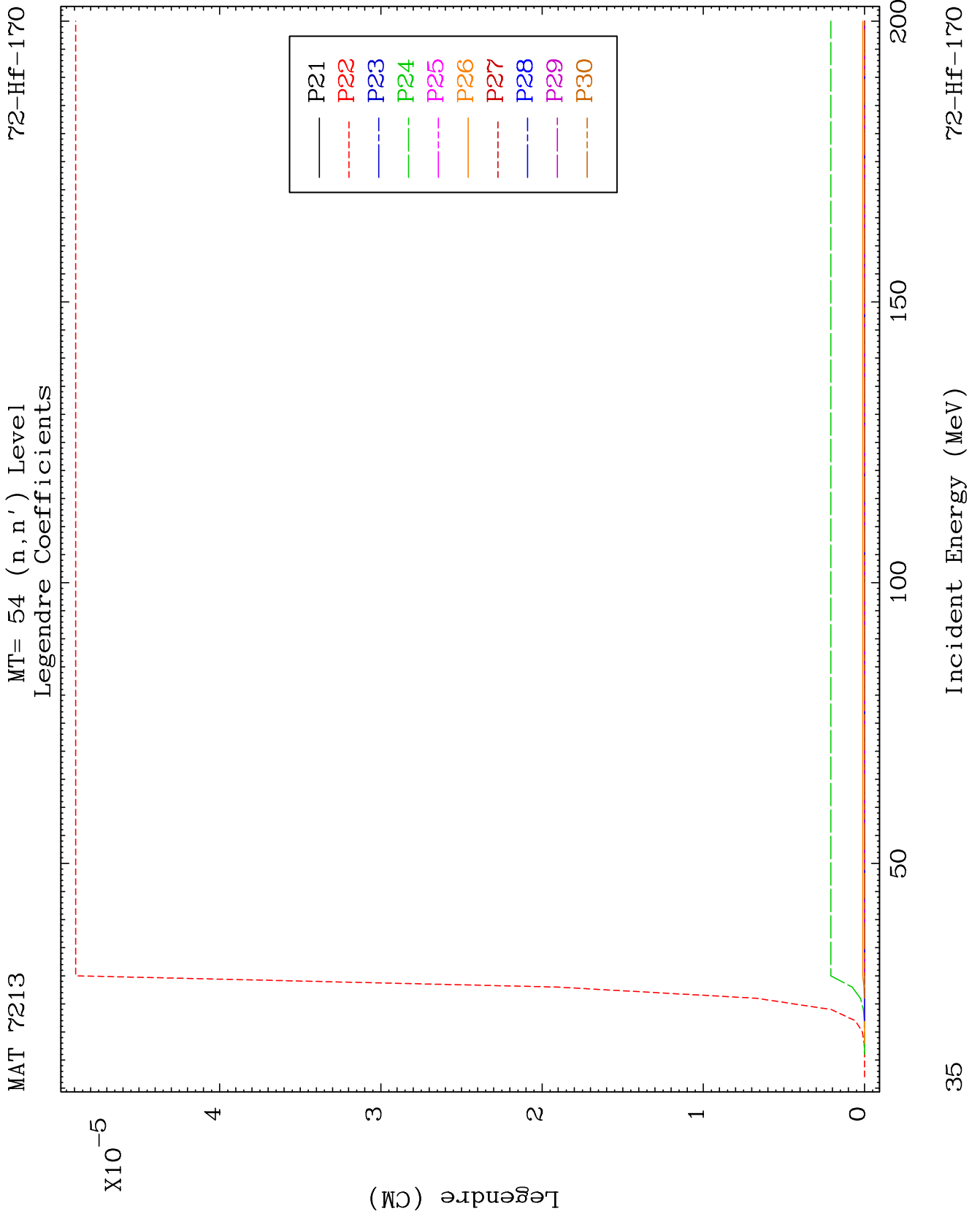
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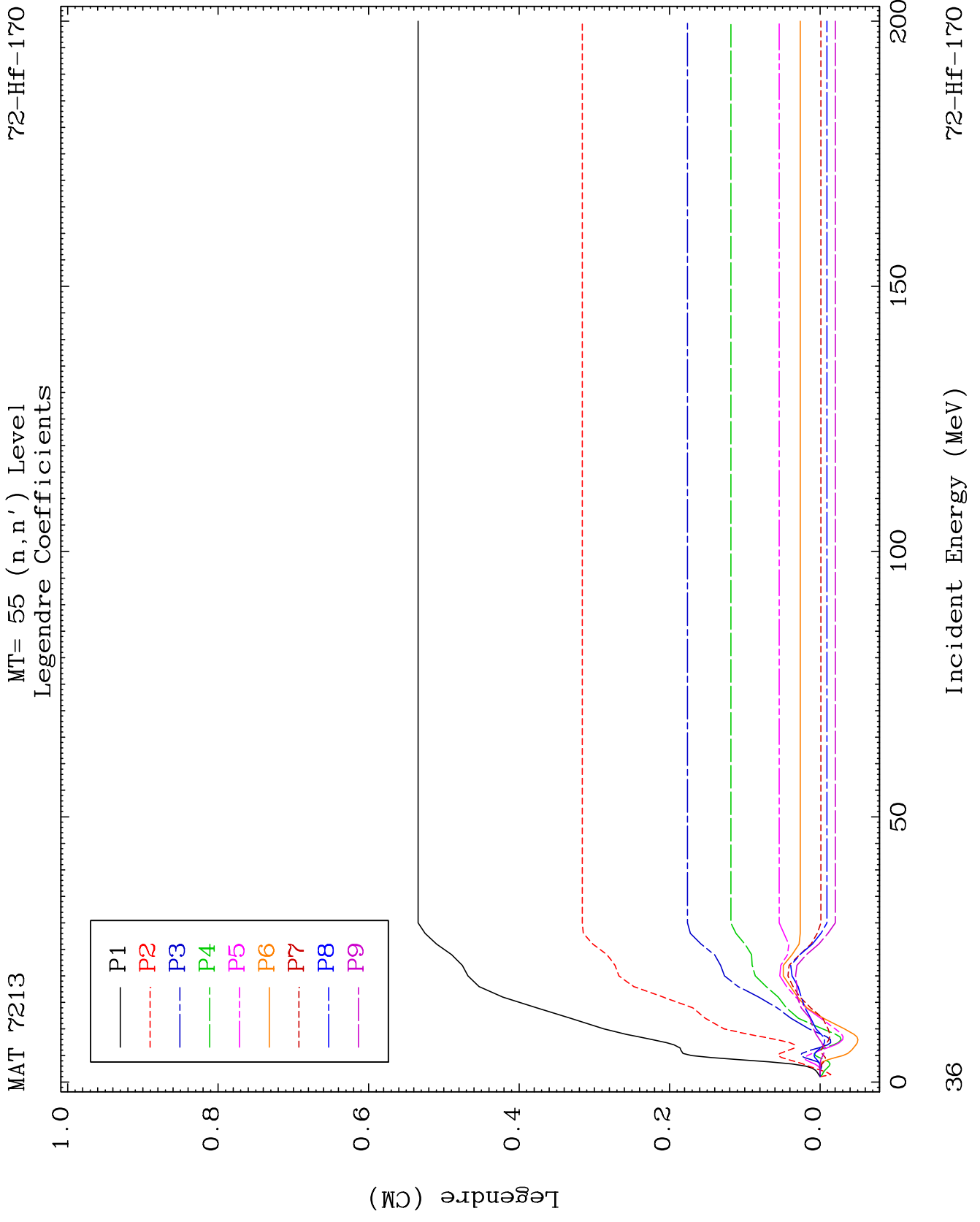


34

Incident Energy (MeV)

72-Hf-170

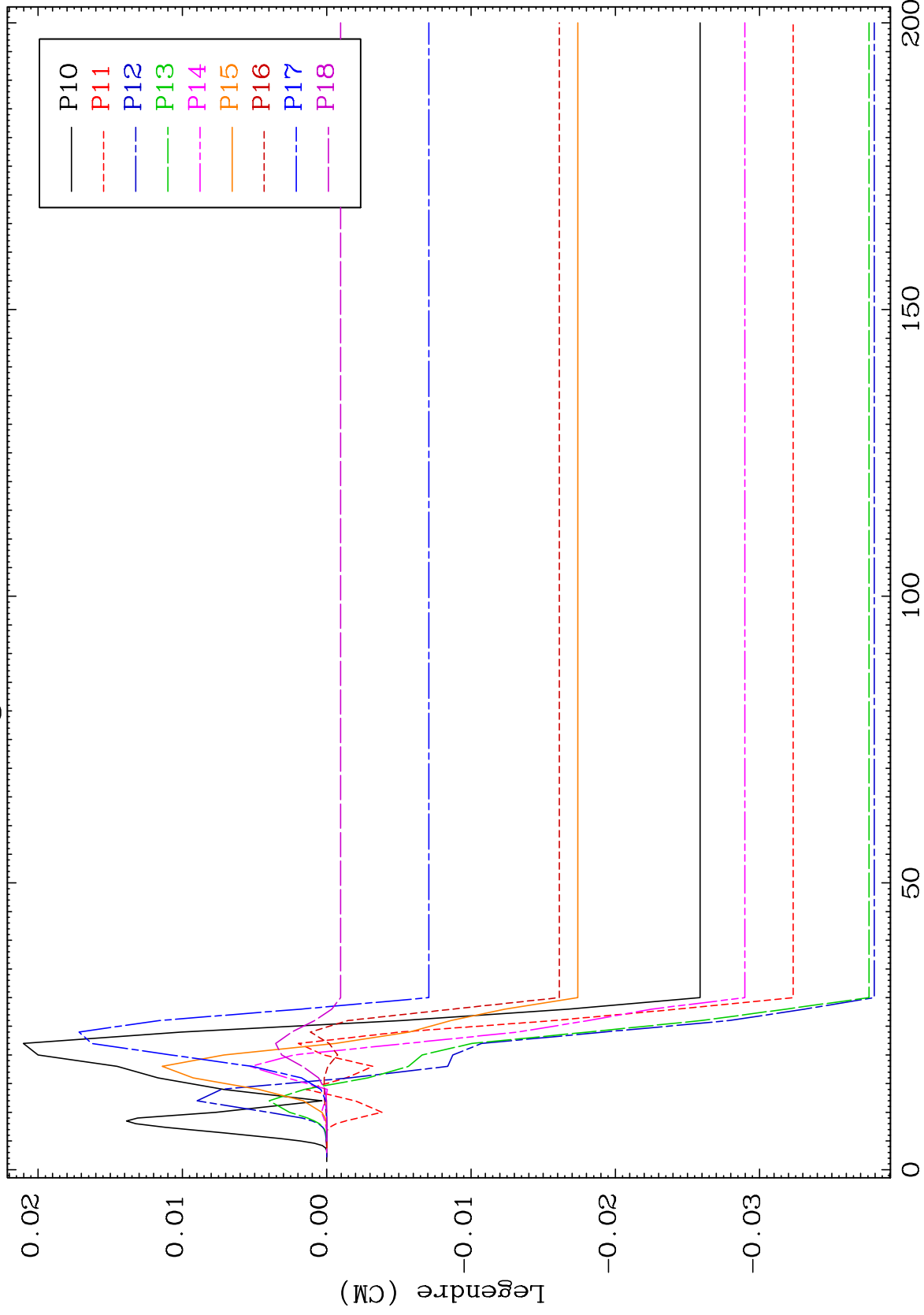




MAT 7213

MT= 55 (n,n') Level
Legendre Coefficients

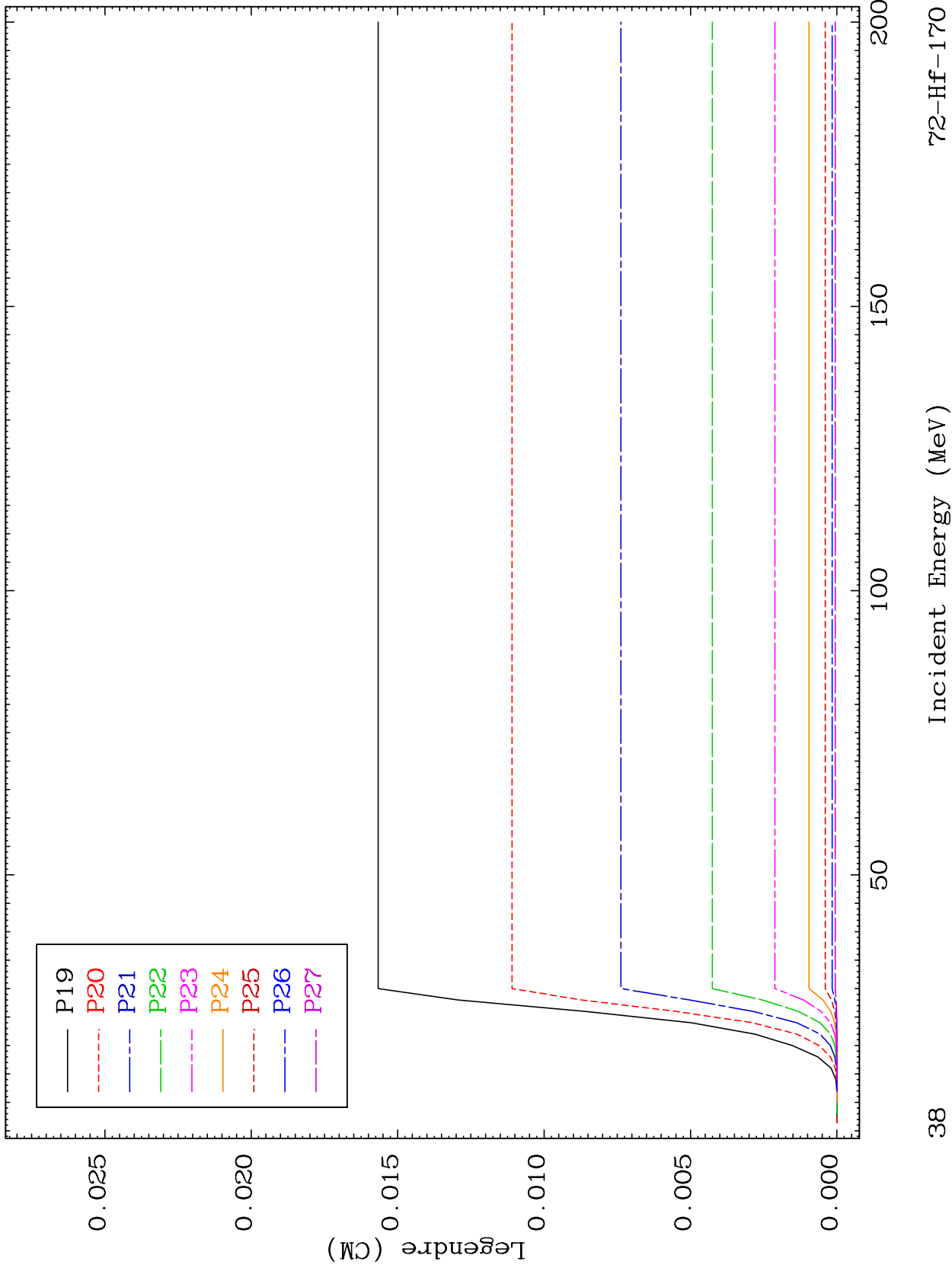
72-Hf-170



37

Incident Energy (MeV)

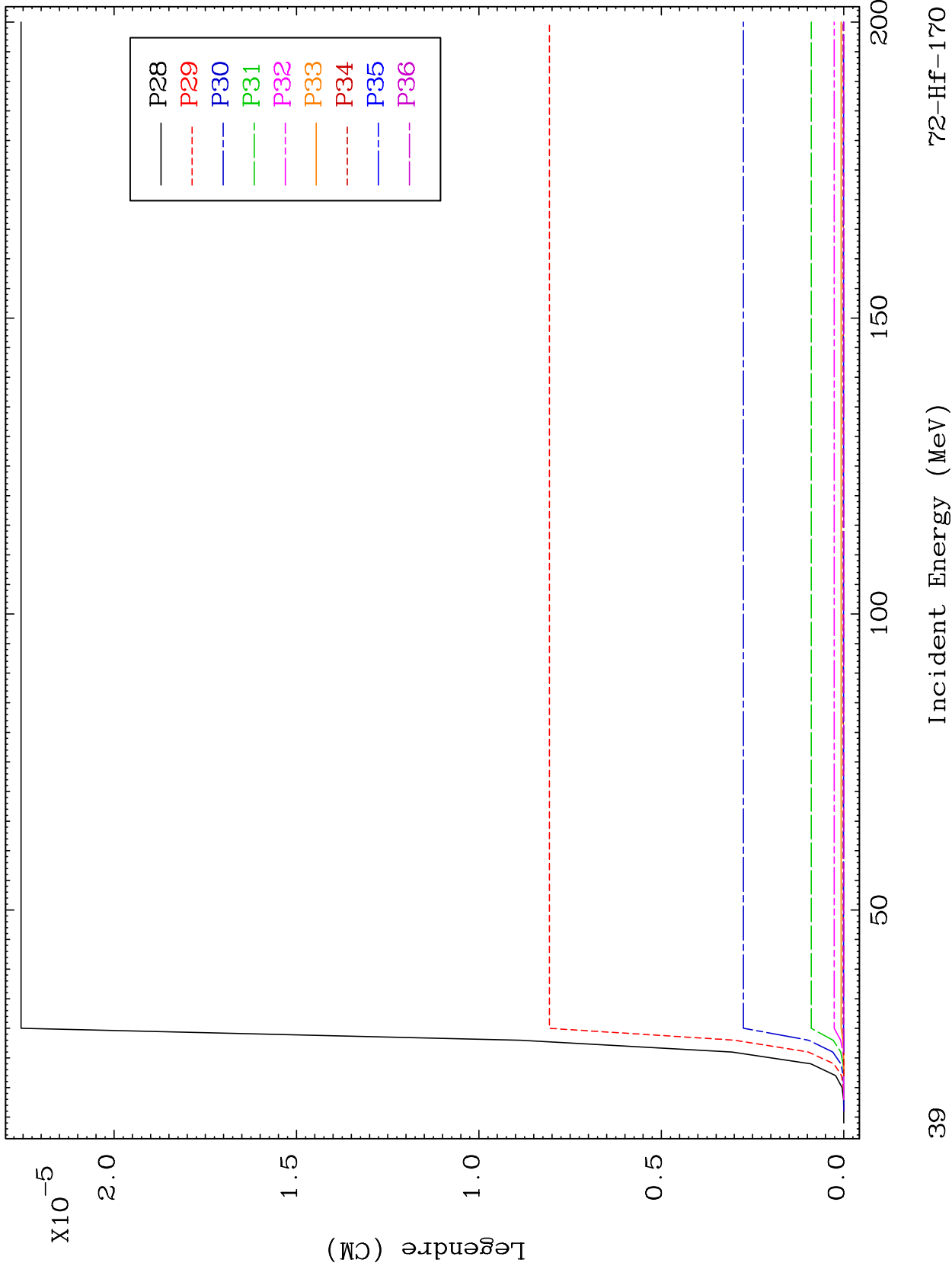
72-Hf-170



MAT 7213

MT= 55 (n,n') Level
Legendre Coefficients

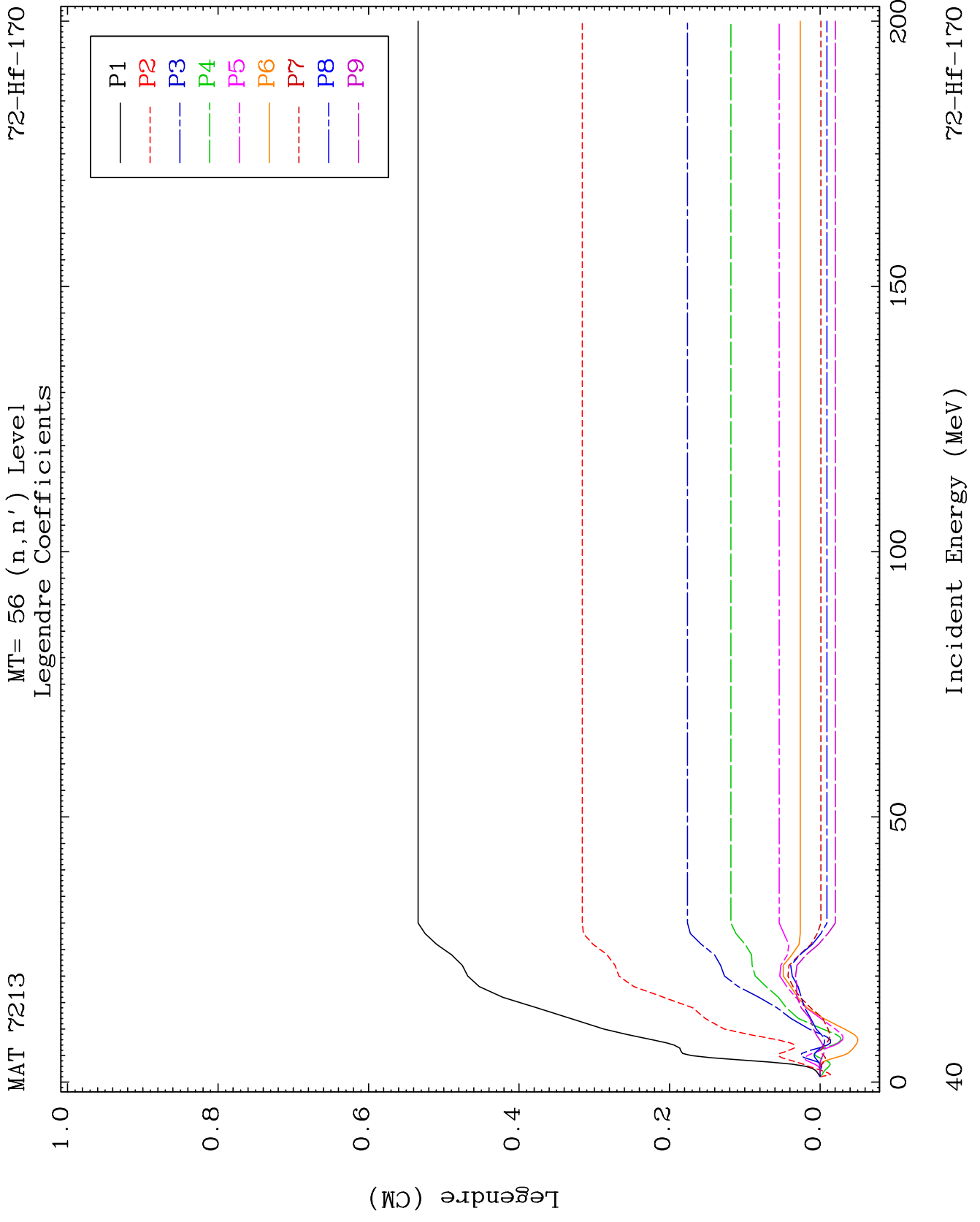
72-Hf-170



39

Incident Energy (MeV)

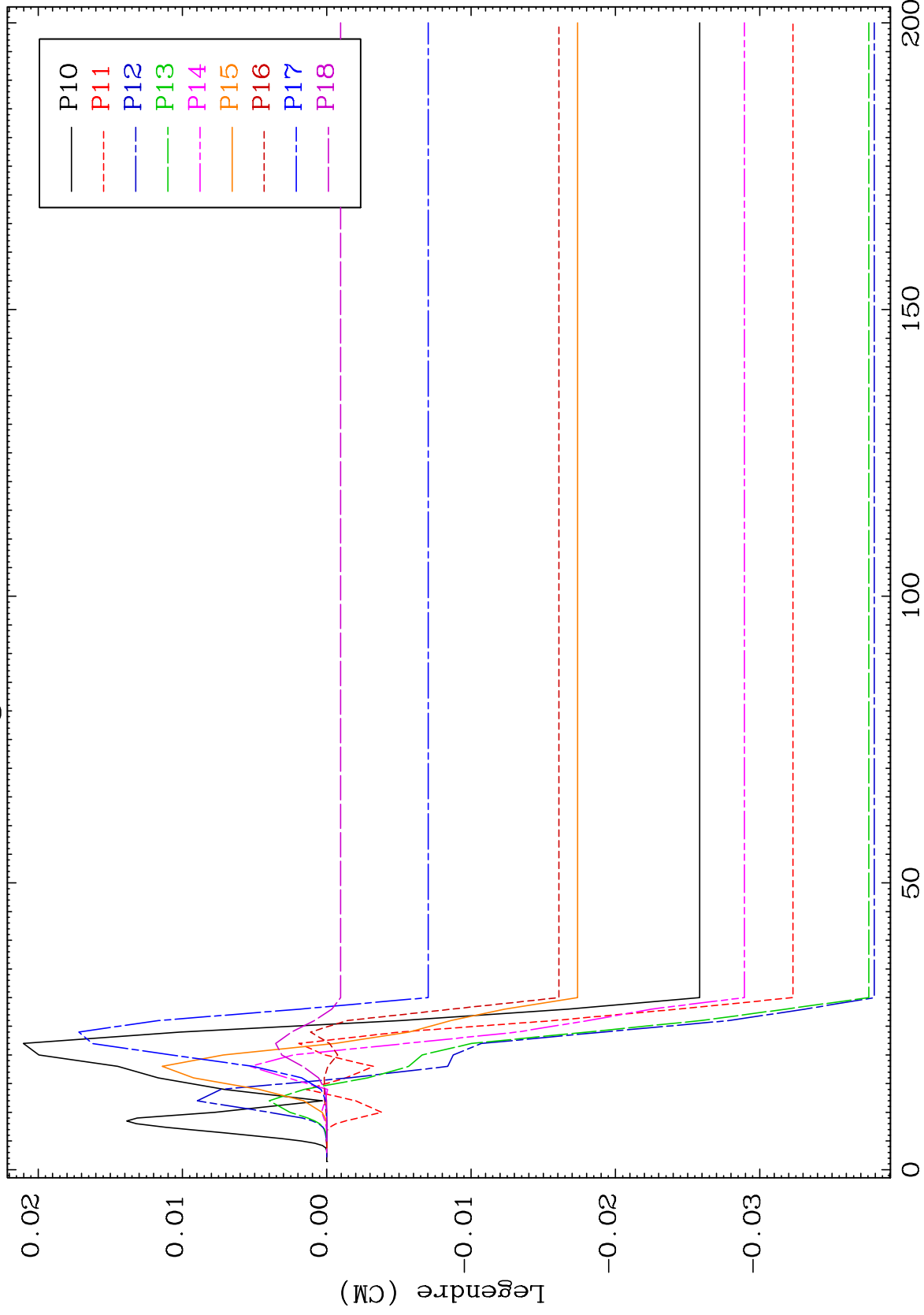
72-Hf-170



MAT 7213

MT= 56 (n,n') Level
Legendre Coefficients

72-Hf-170



41

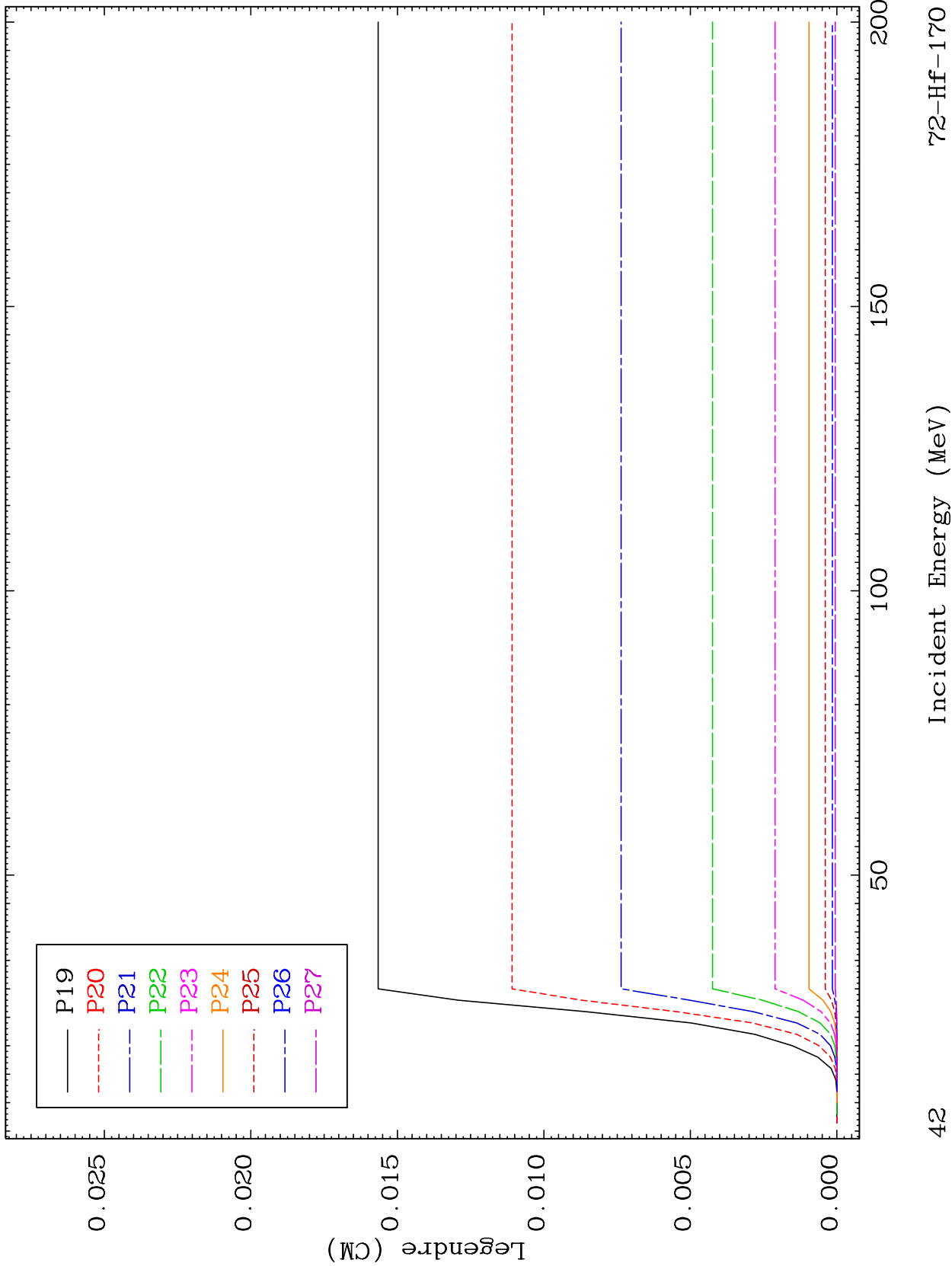
Incident Energy (MeV)

72-Hf-170

MAT 7213

MT= 56 (n,n') Level
Legendre Coefficients

72-Hf-170



42

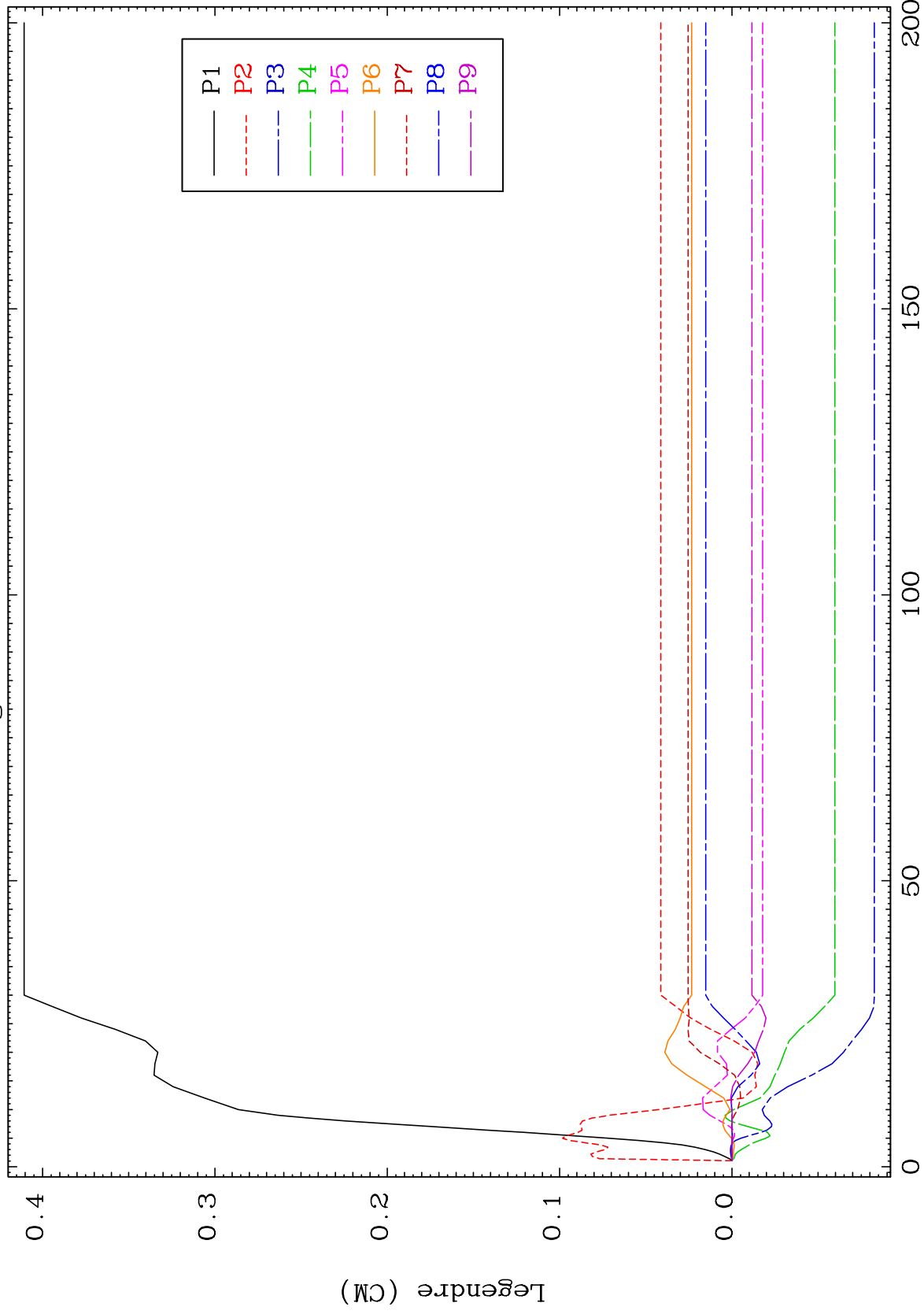
Incident Energy (MeV)

72-Hf-170

MAT 7213

MT= 57 (n,n') Level
Legendre Coefficients

72-Hf-170



44

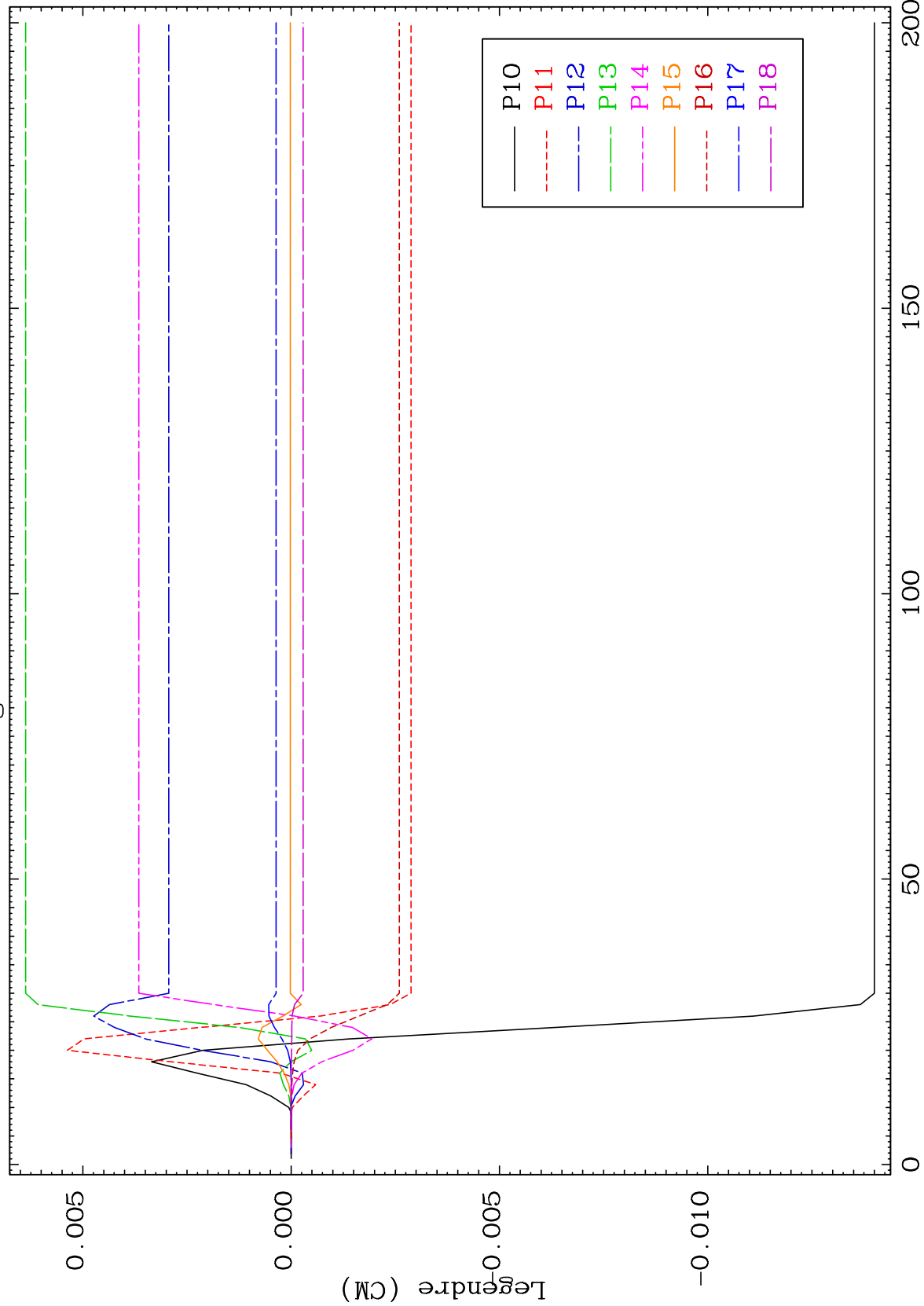
Incident Energy (MeV)

72-Hf-170

MAT 7213

MT= 57 (n,n') Level
Legendre Coefficients

72-Hf-170



45

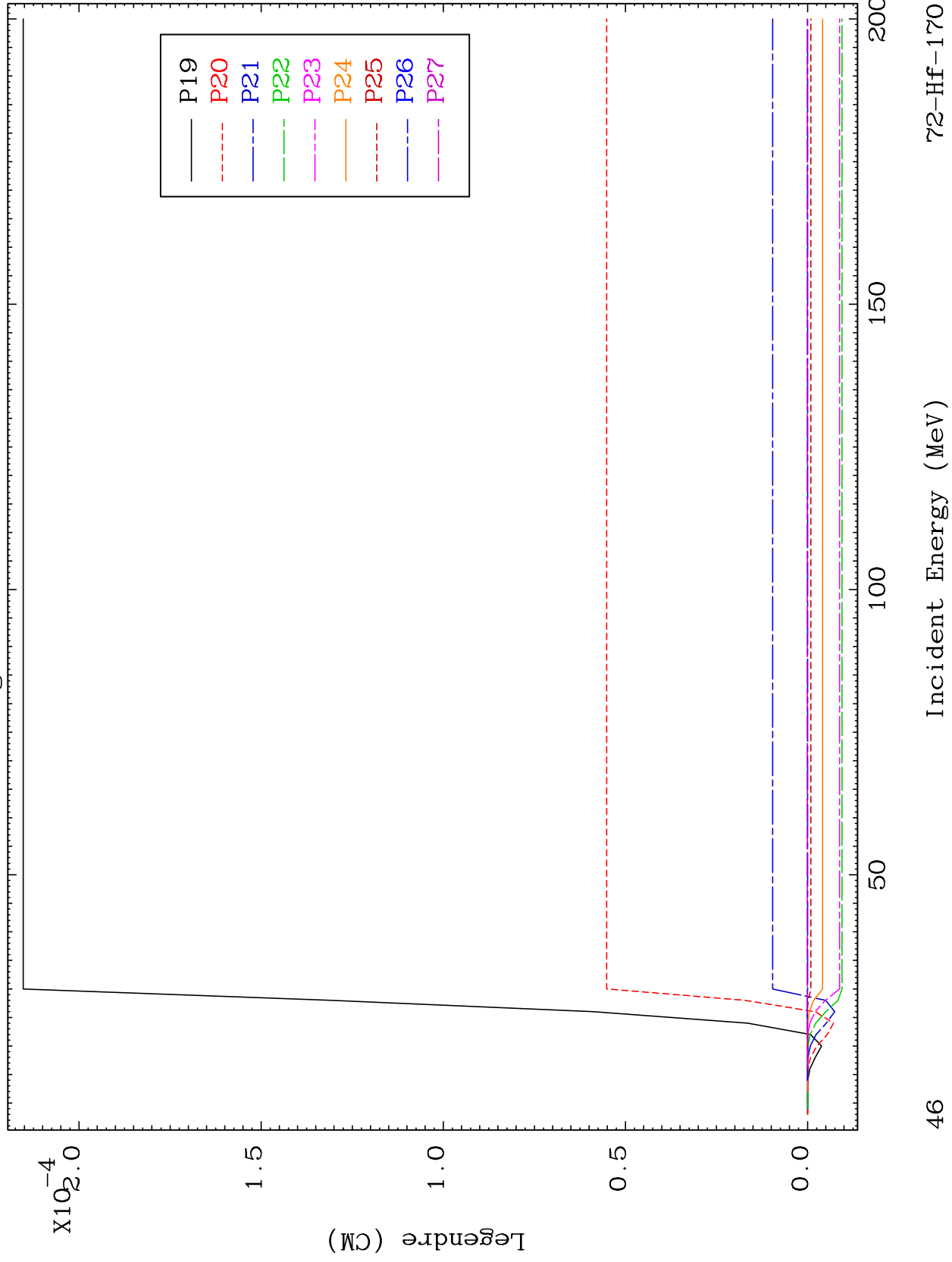
Incident Energy (MeV)

72-Hf-170

MAT 7213

MT= 57 (n,n') Level
Legendre Coefficients

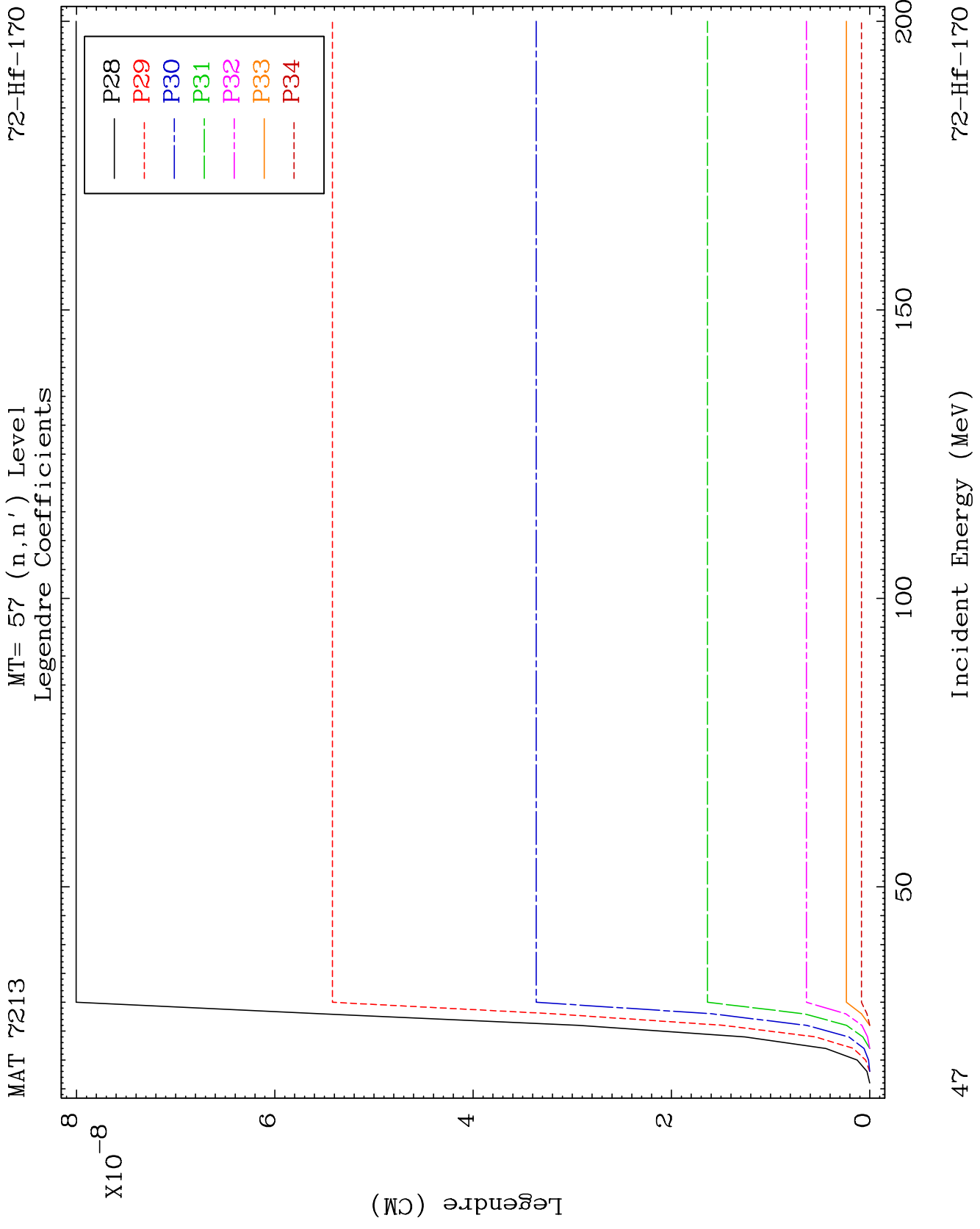
72-Hf-170



46

Incident Energy (MeV)

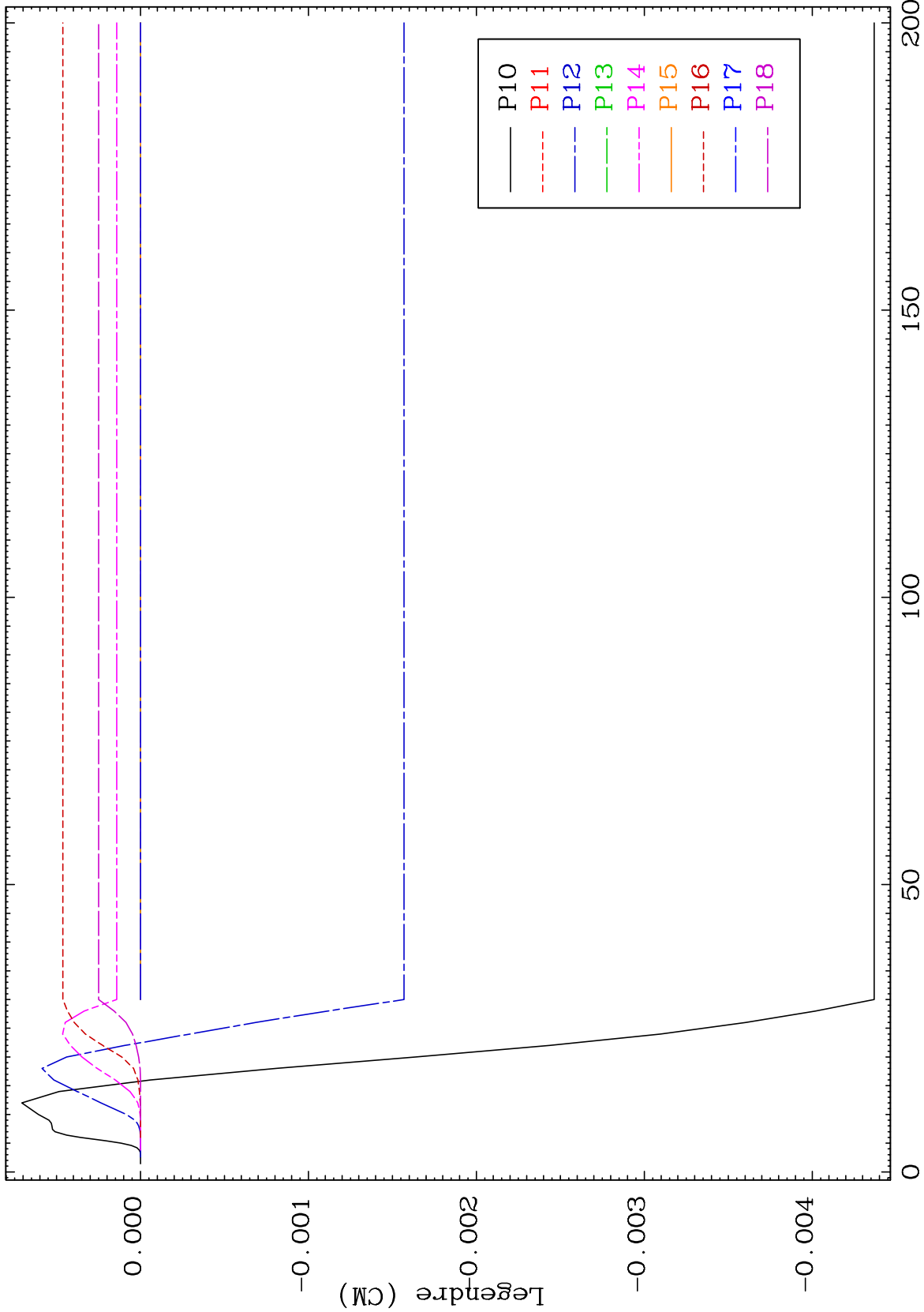
72-Hf-170



MAT 7213

MT= 58 (n,n') Level
Legendre Coefficients

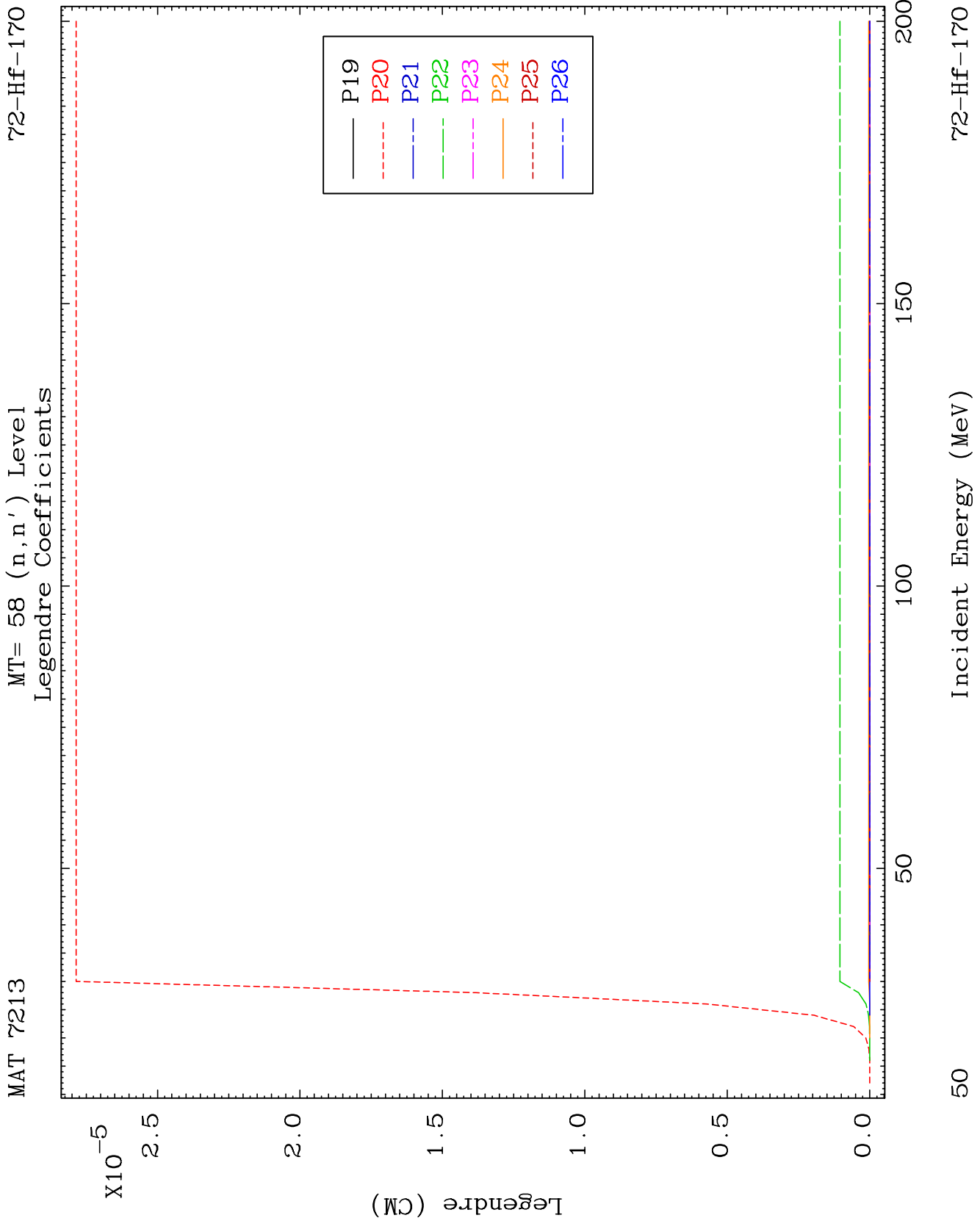
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49

Incident Energy (MeV)

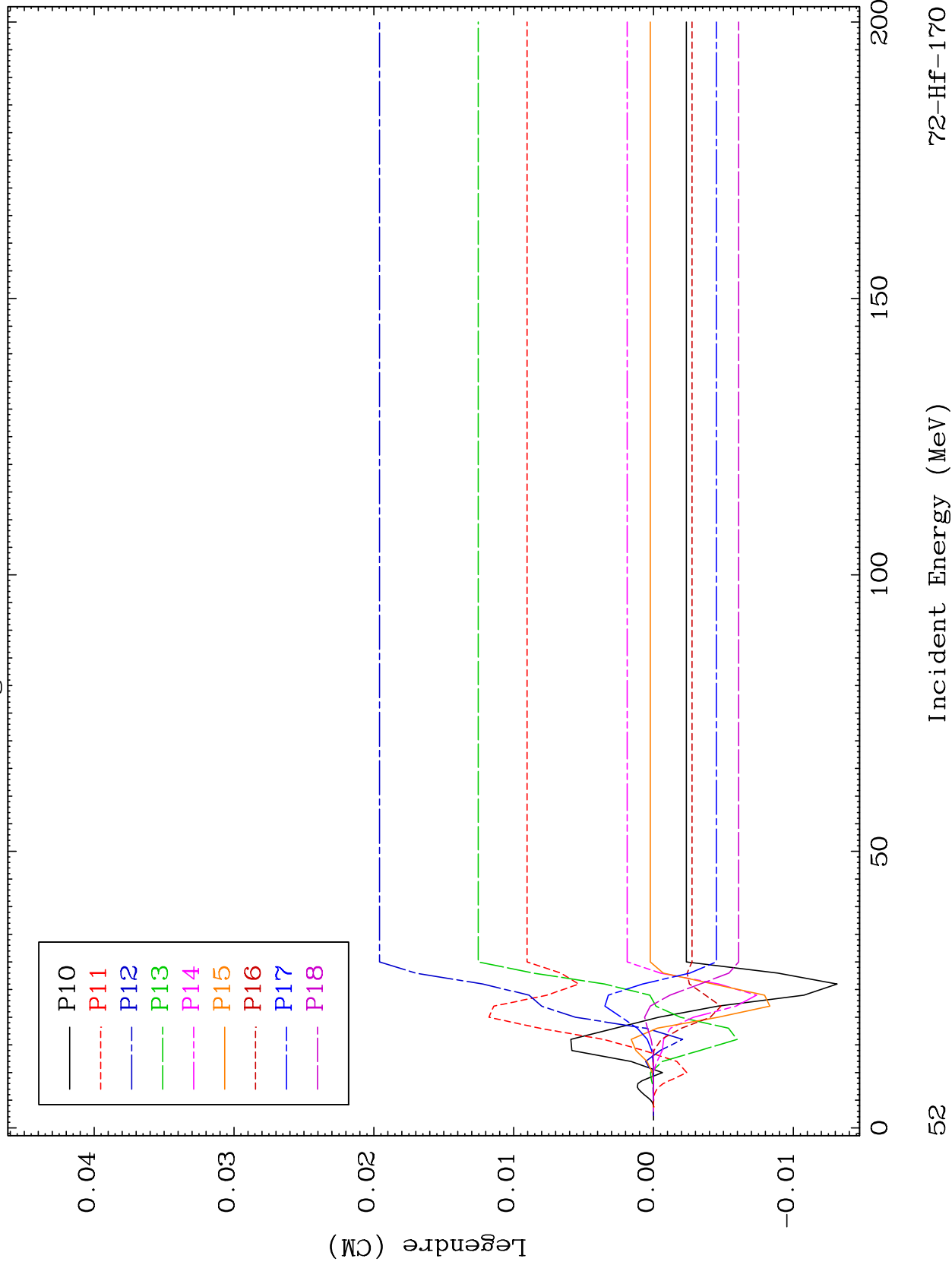
72-Hf-170



MAT 7213

MT= 59 (n,n') Level
Legendre Coefficients

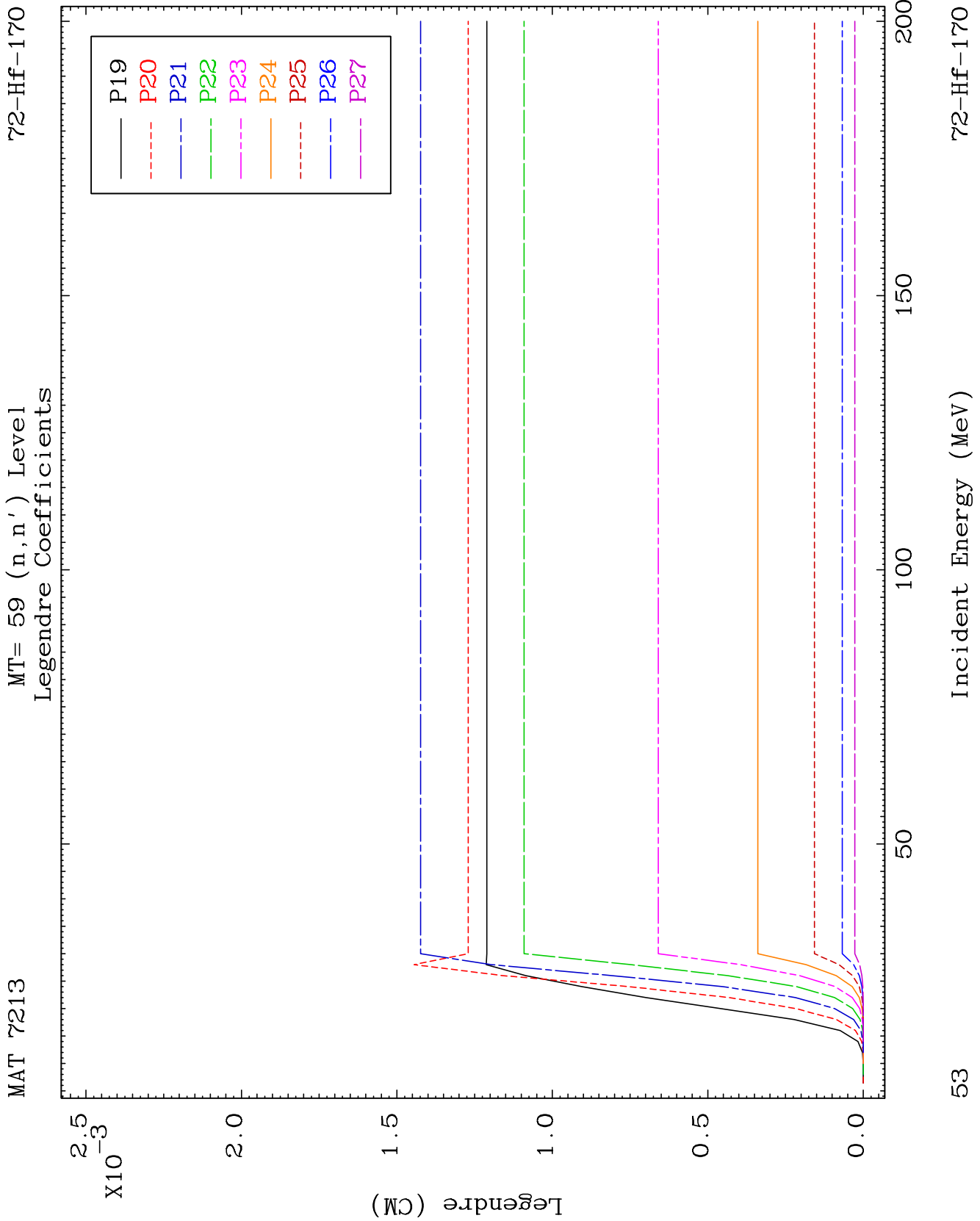
72-Hf-170



72-Hf-170

Incident Energy (MeV)

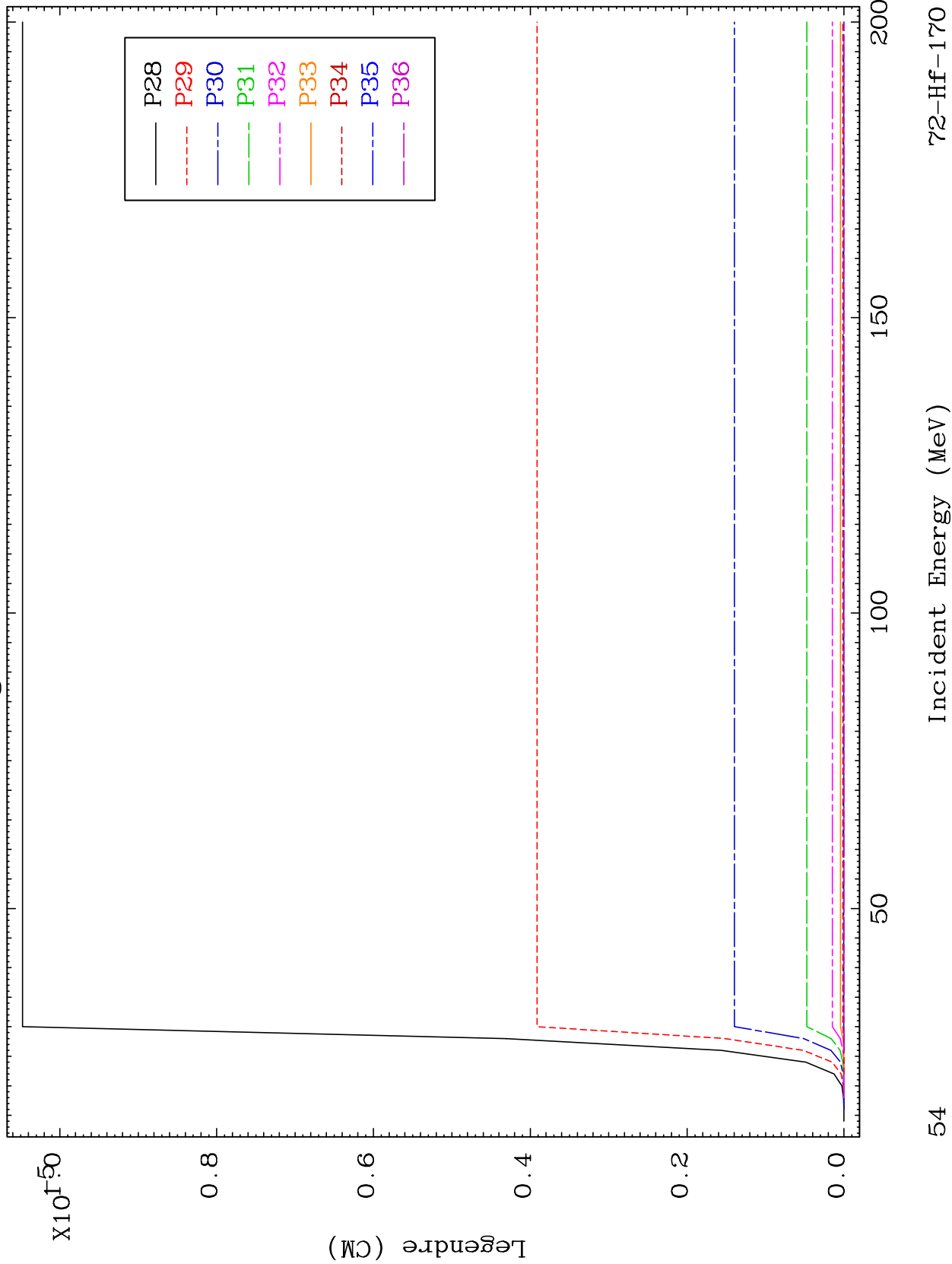
52



MAT 7213

MT= 59 (n,n') Level
Legendre Coefficients

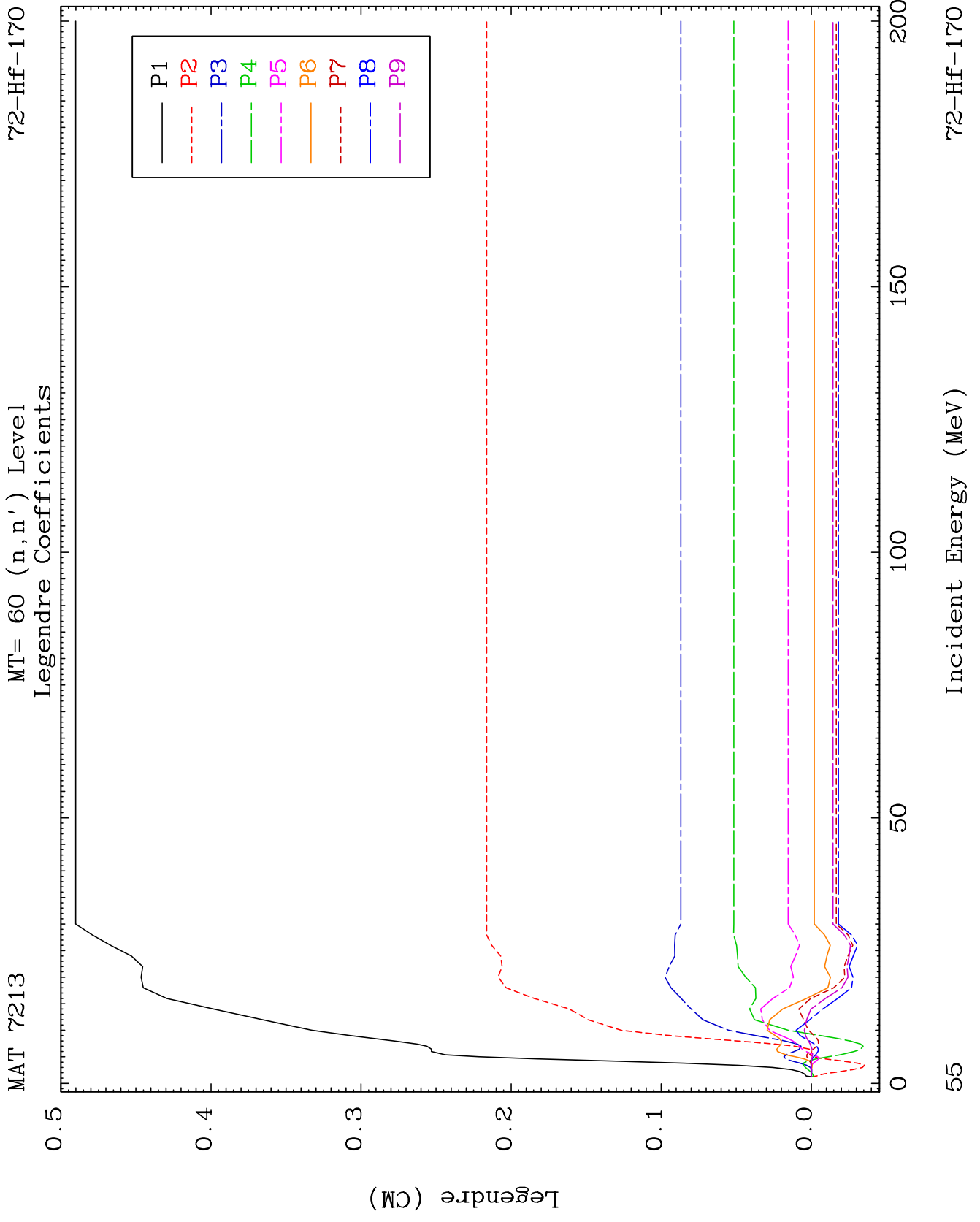
72-Hf-170



54

Incident Energy (MeV)

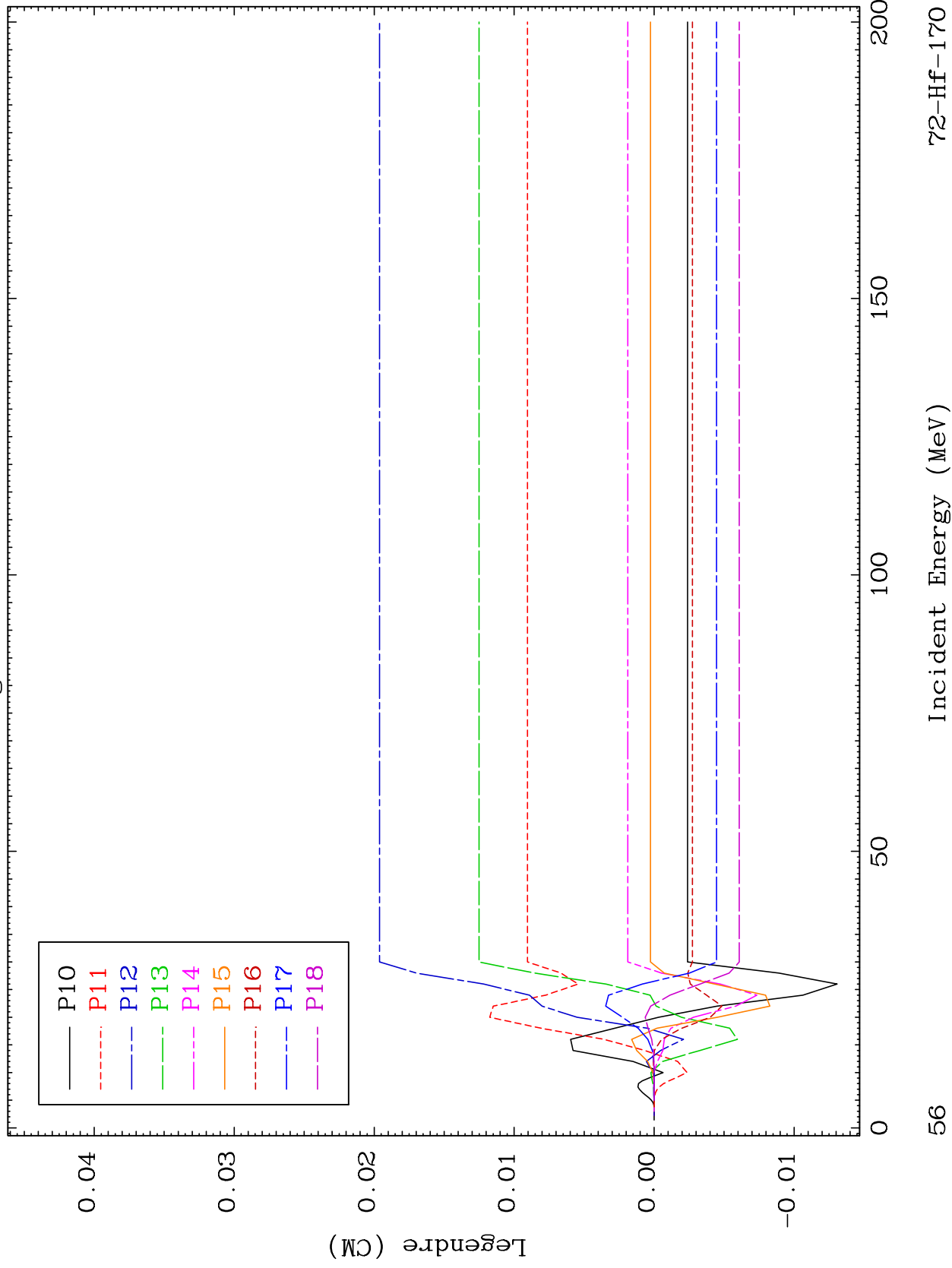
72-Hf-170



MAT 7213

MT= 60 (n,n') Level
Legendre Coefficients

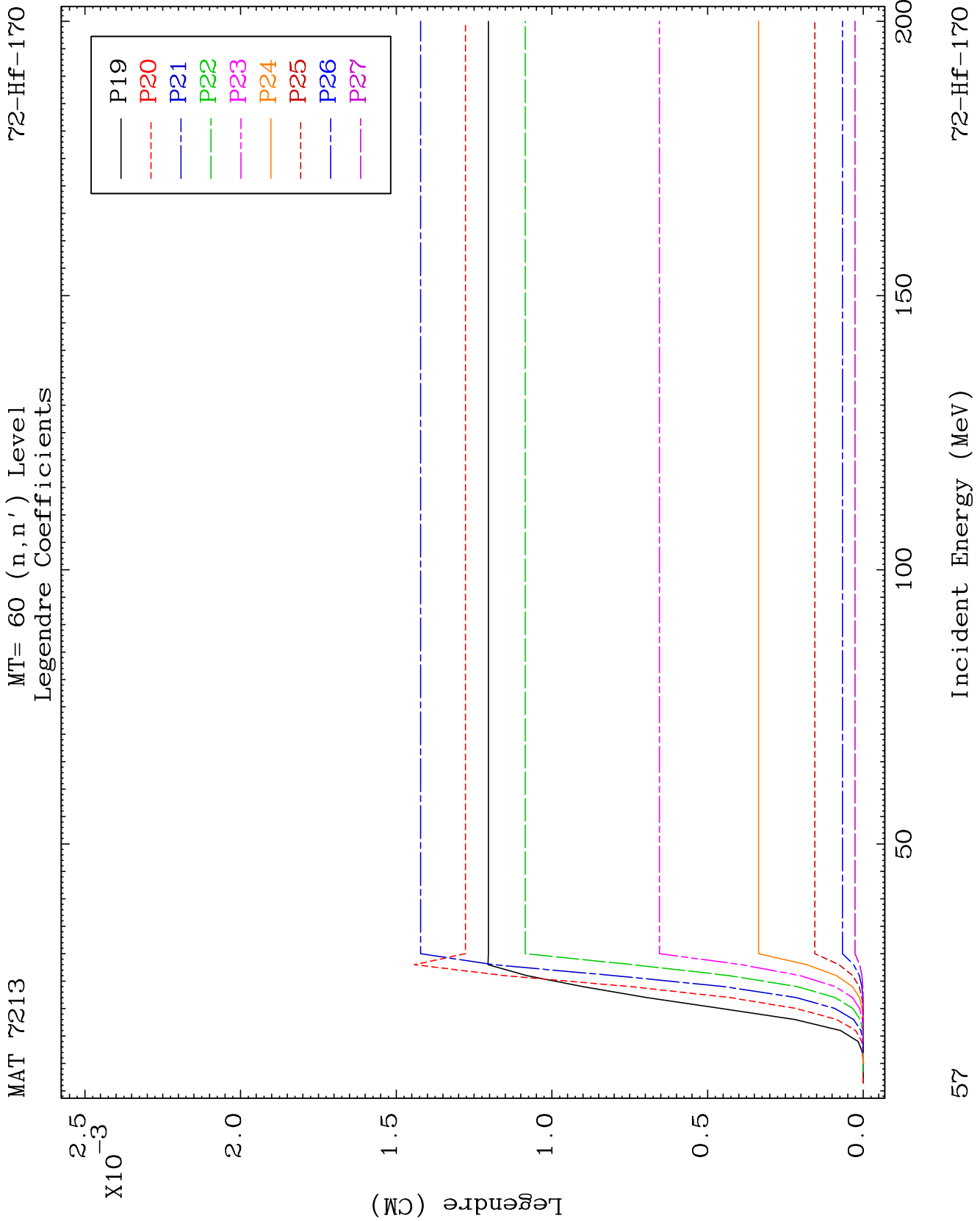
72-Hf-170



56

Incident Energy (MeV)

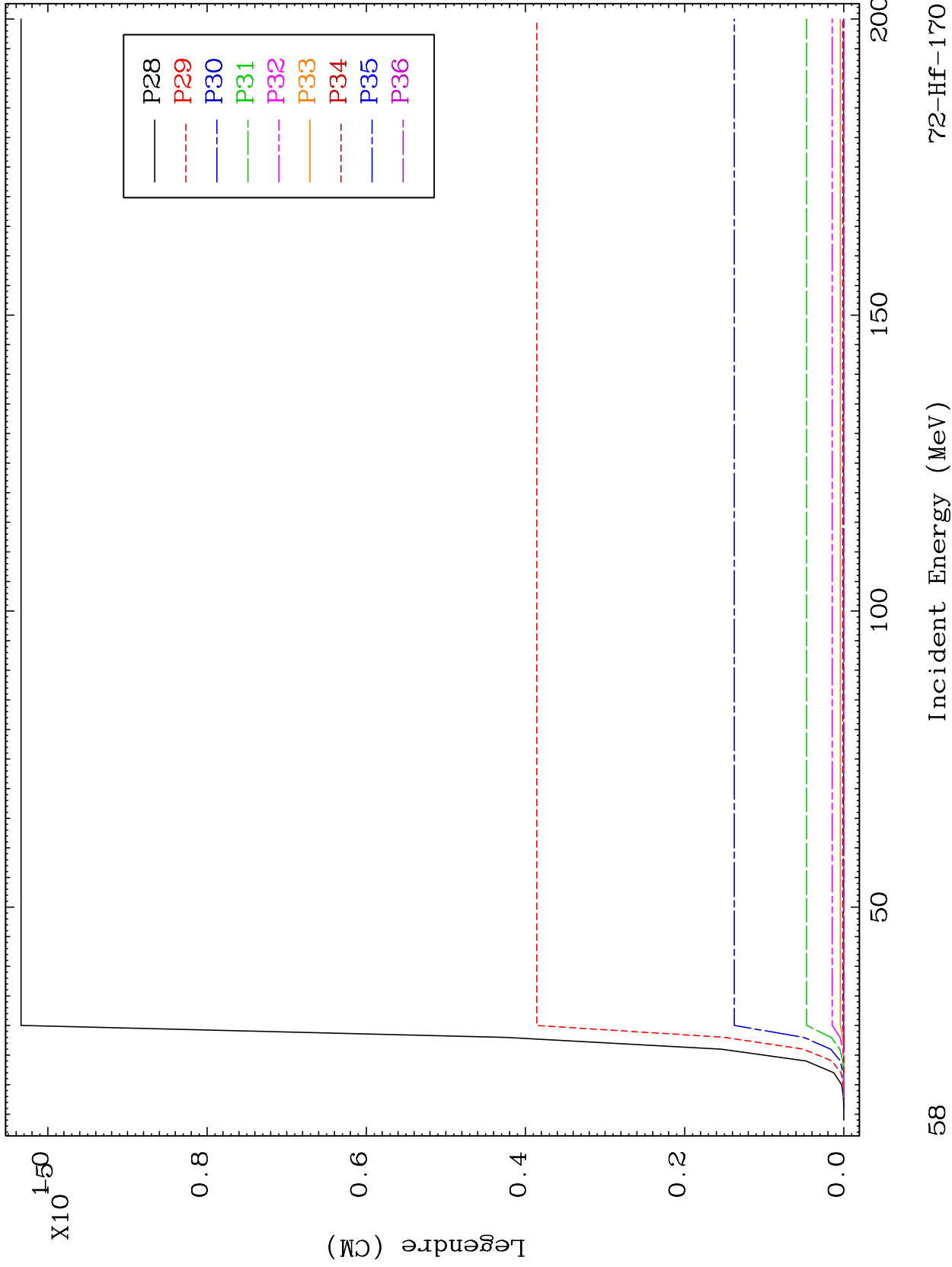
72-Hf-170



MAT 7213

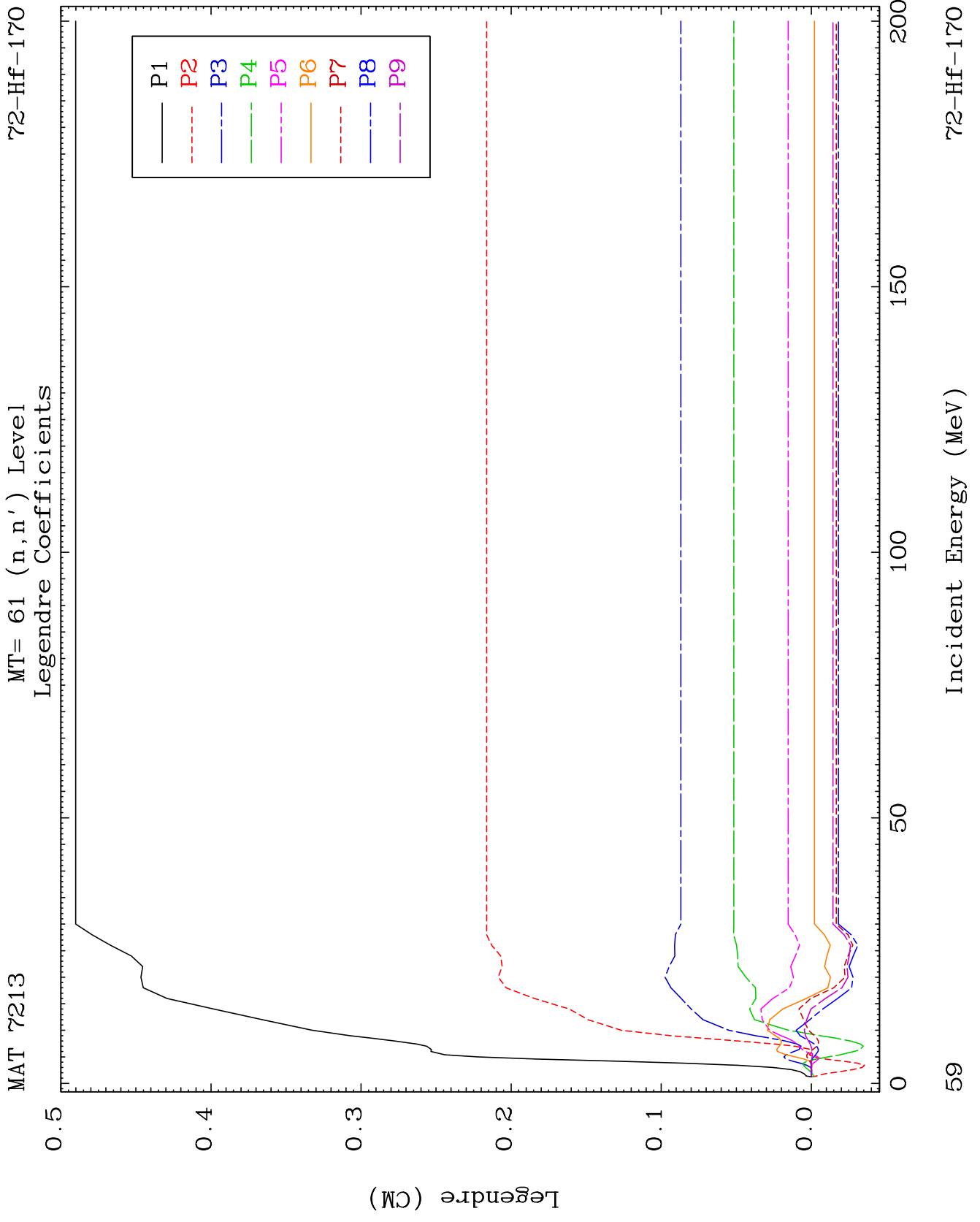
MT= 60 (n,n') Level
Legendre Coefficients

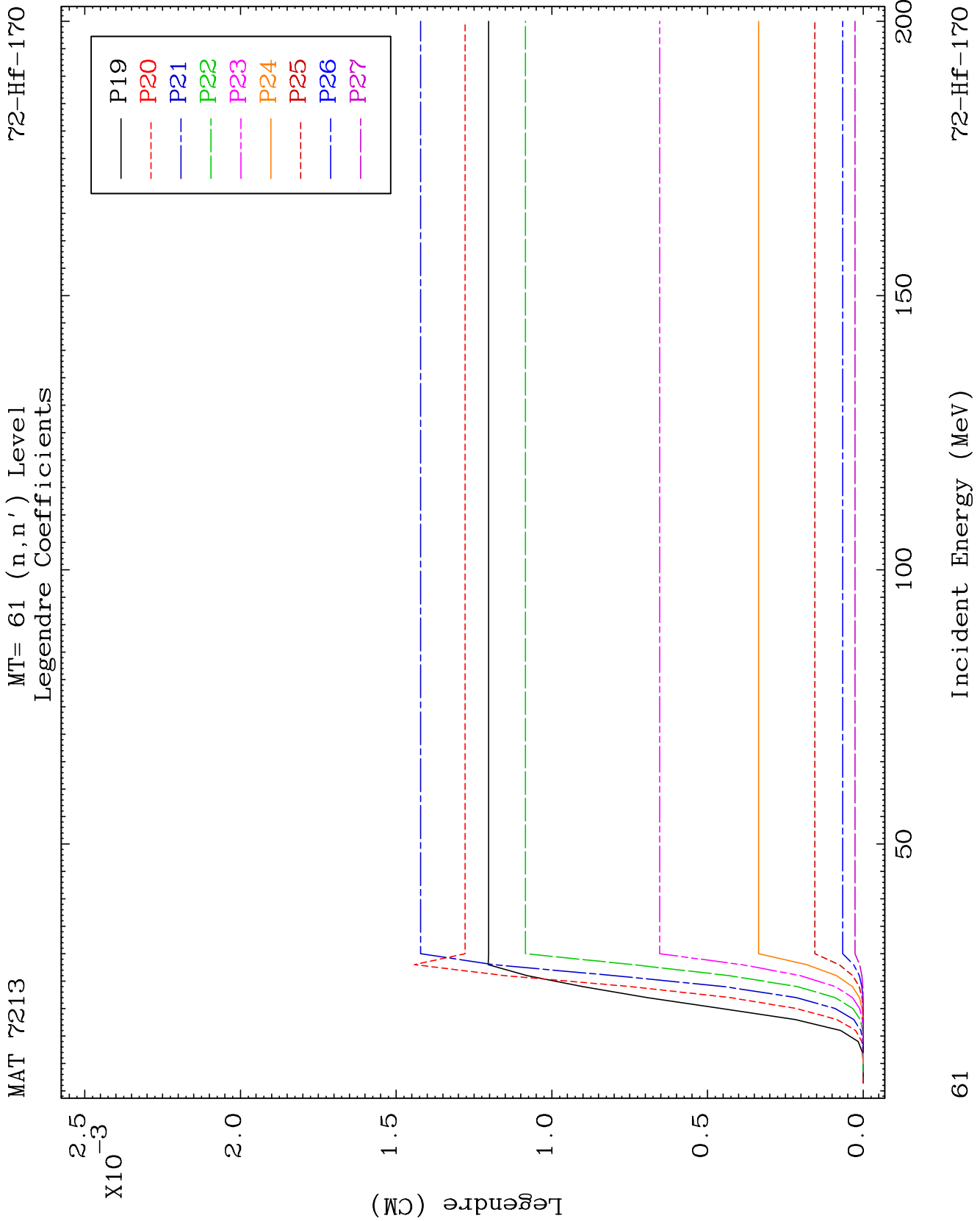
72-Hf-170



58

72-Hf-170

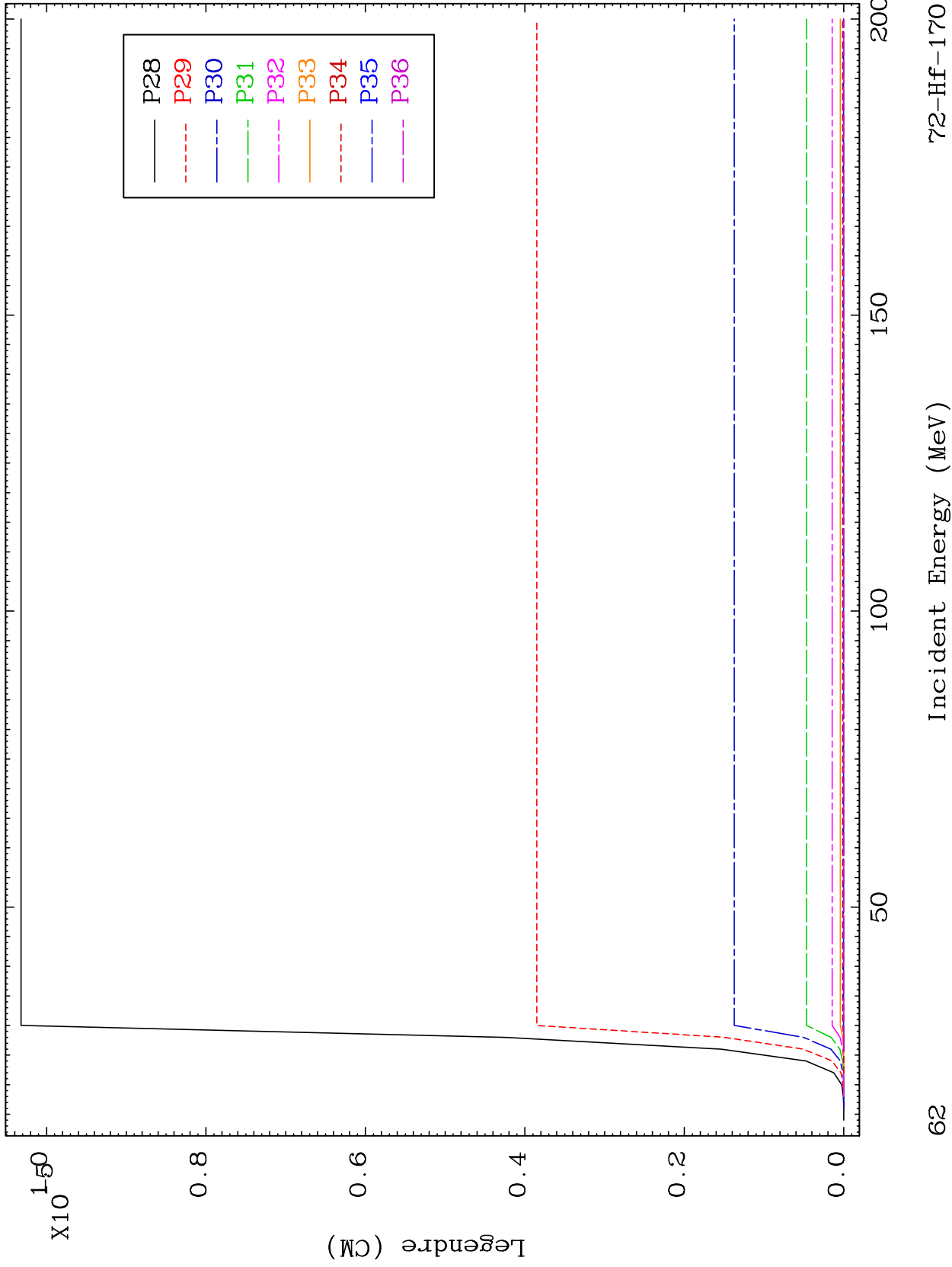




MAT 7213

MT= 61 (n,n') Level
Legendre Coefficients

72-Hf-170



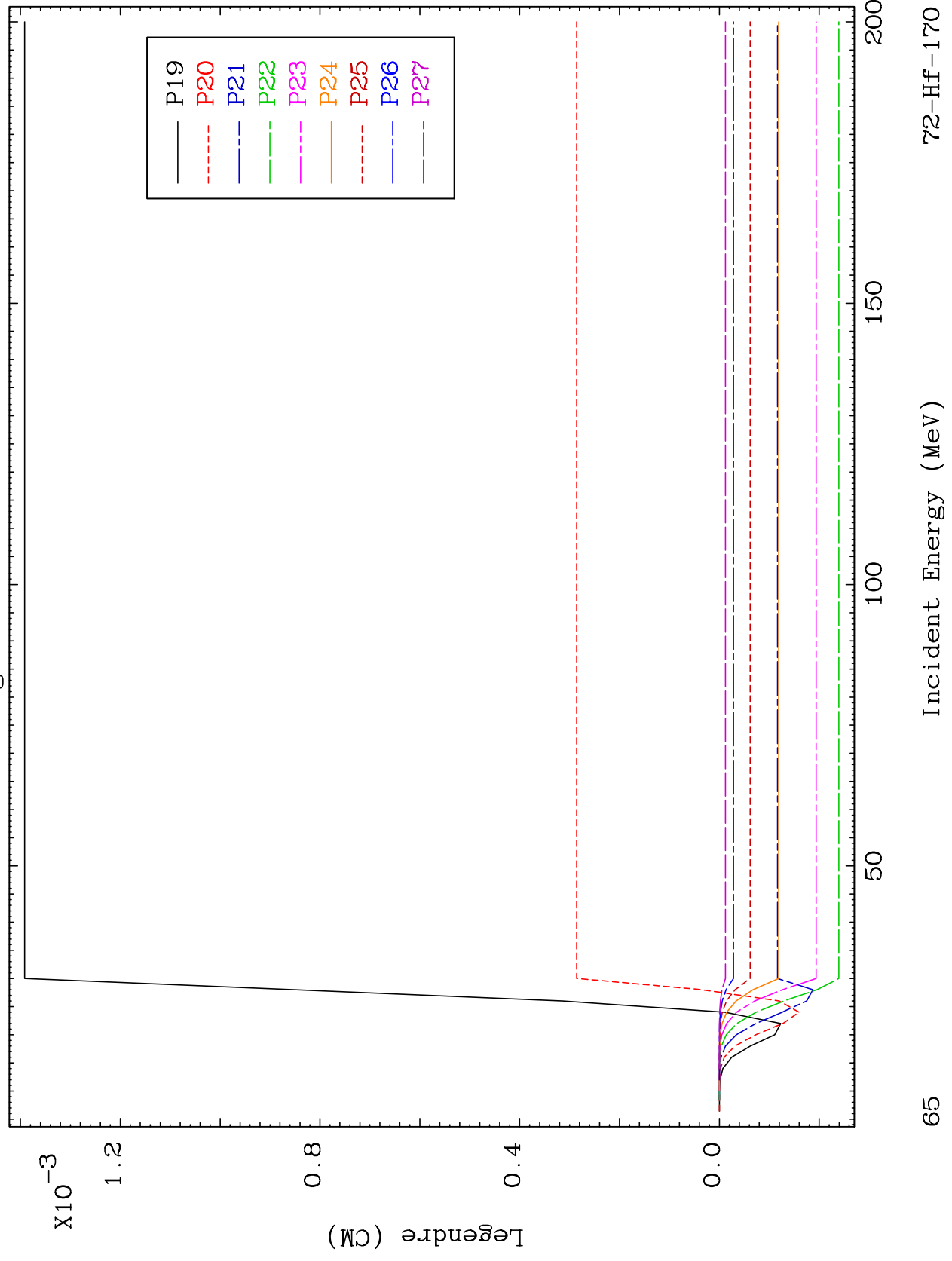
62

72-Hf-170

MAT 7213

MT= 62 (n,n') Level
Legendre Coefficients

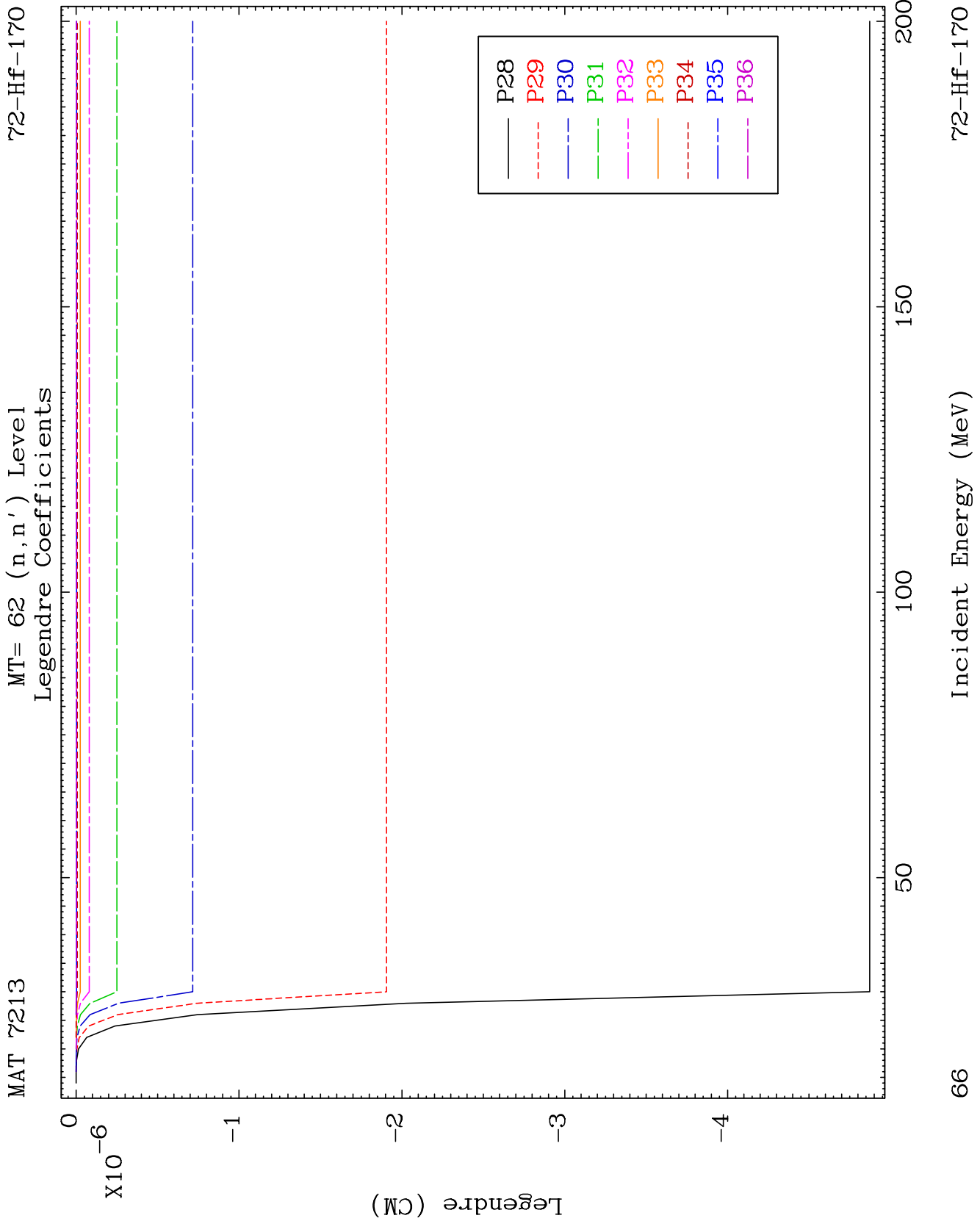
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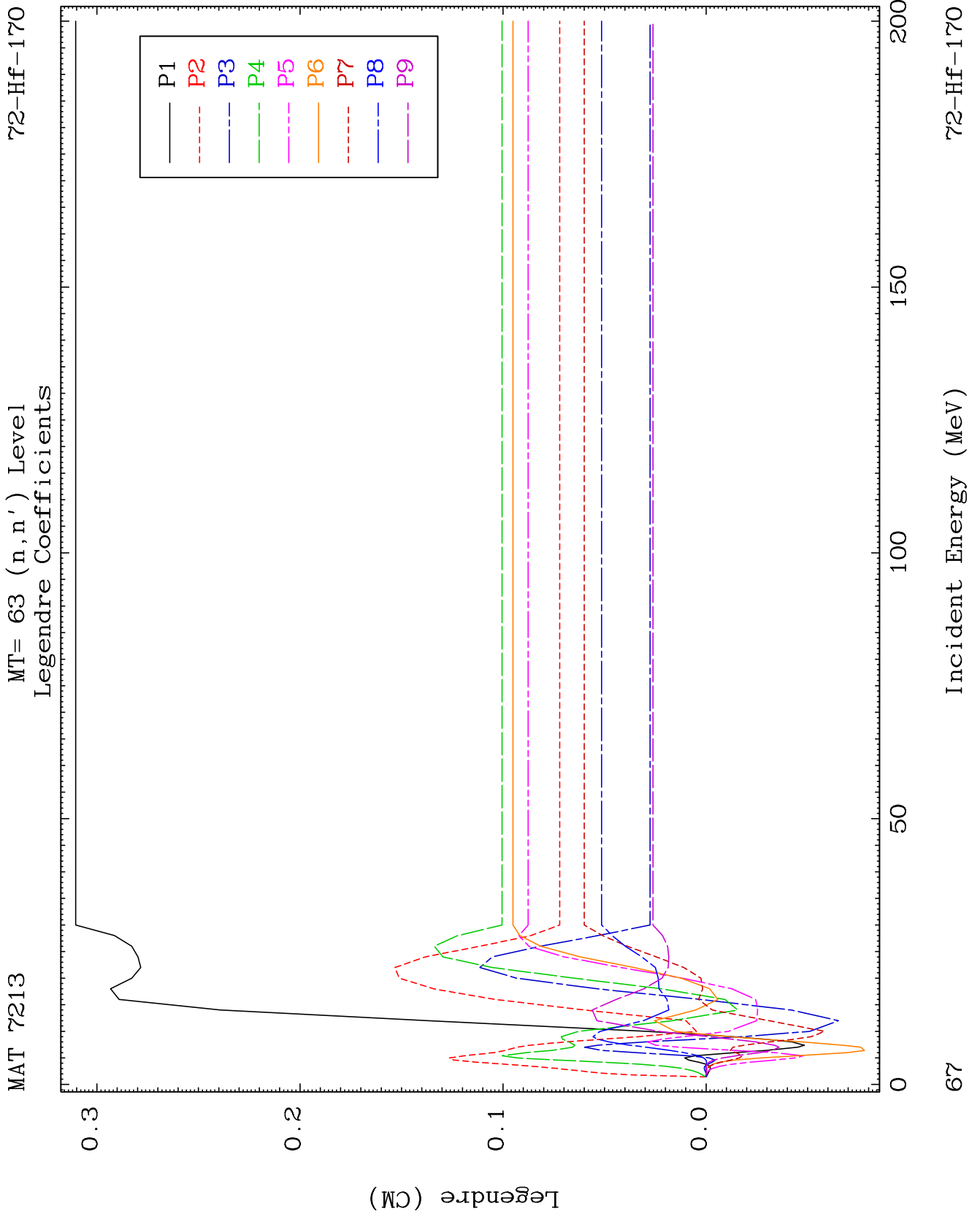


65

Incident Energy (MeV)

72-Hf-170

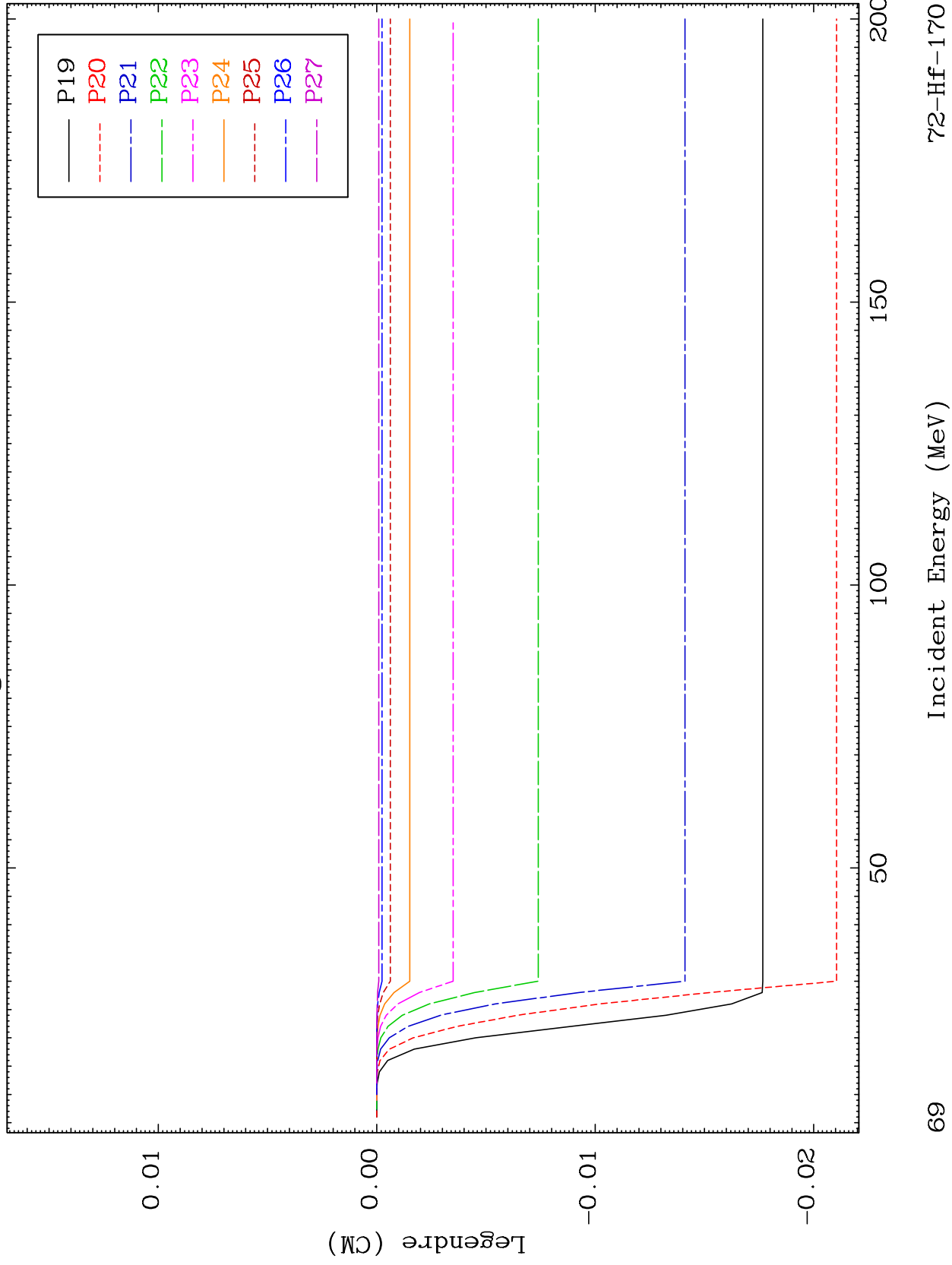




MAT 7213

MT= 63 (n,n') Level
Legendre Coefficients

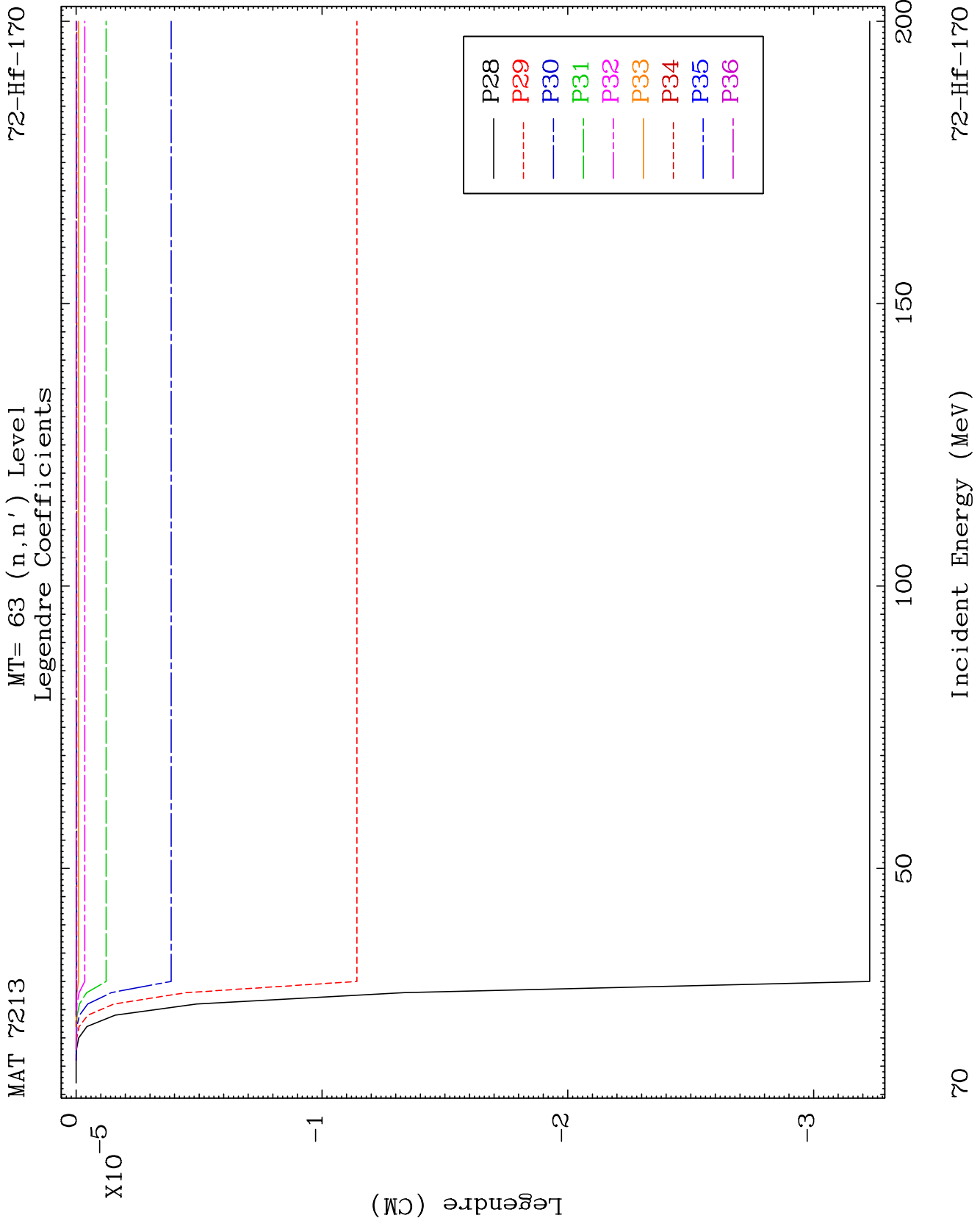
72-Hf-170



69

Incident Energy (MeV)

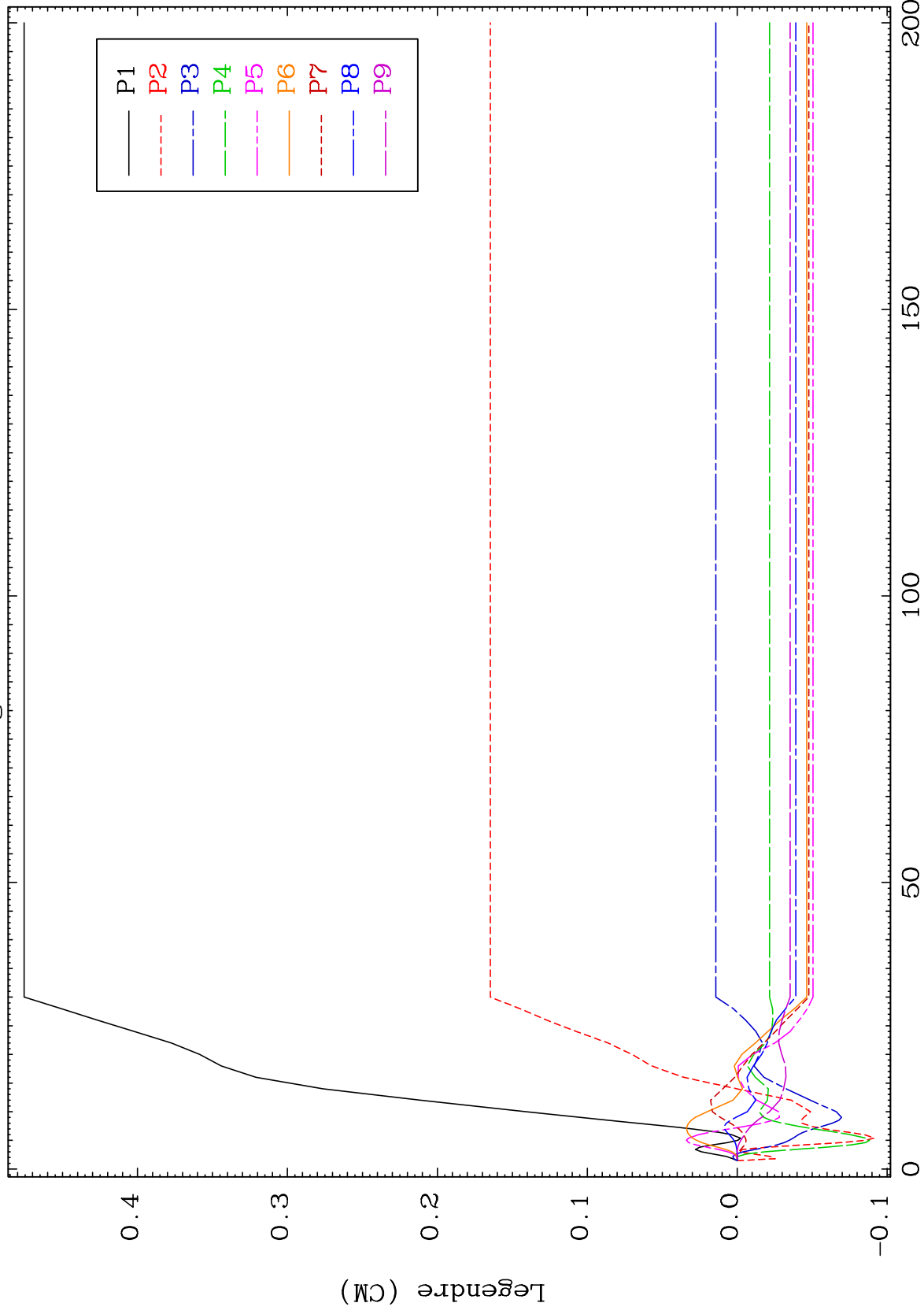
72-Hf-170



MAT 7213

MT= 64 (n,n') Level
Legendre Coefficients

72-Hf-170



71

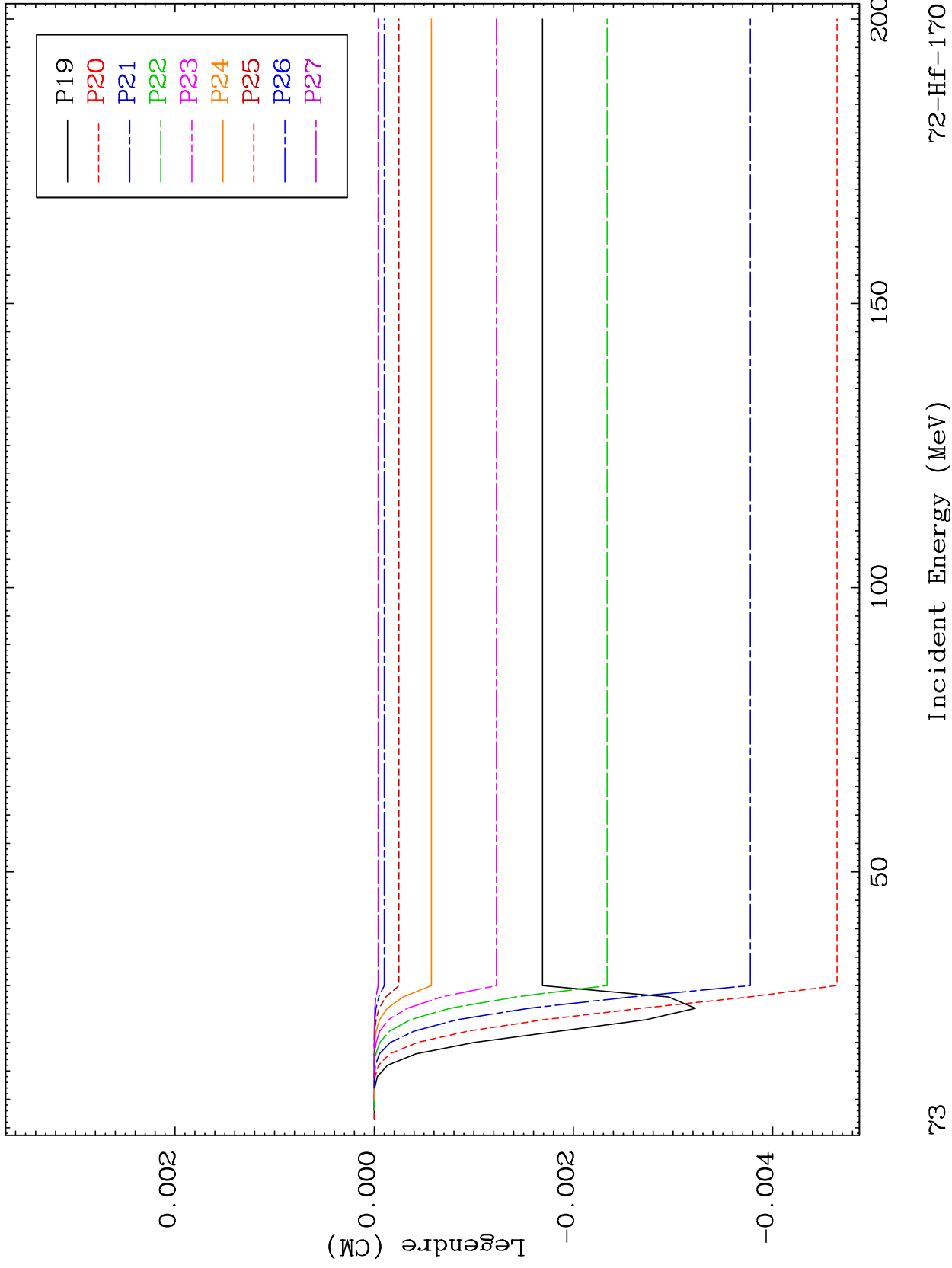
Incident Energy (MeV)

72-Hf-170

MAT 7213

MT= 64 (n,n') Level
Legendre Coefficients

72-Hf-170



73

Incident Energy (MeV)

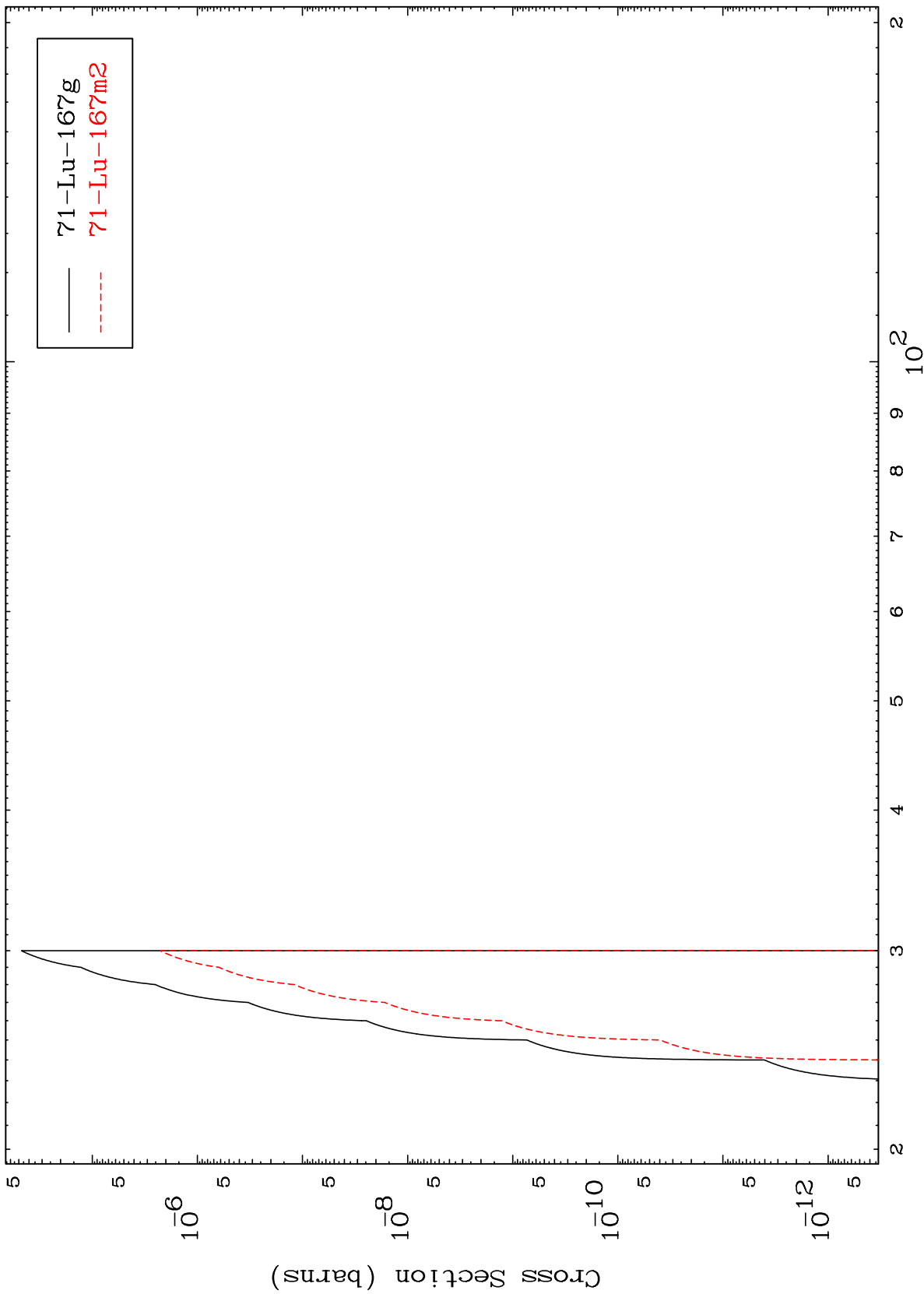
72-Hf-170

MAT 7213

(n,2n) d

72-Hf-170

Radionuclide Production Cross Section



71-Lu-167g
71-Lu-167m2

75

Incident Energy (MeV)

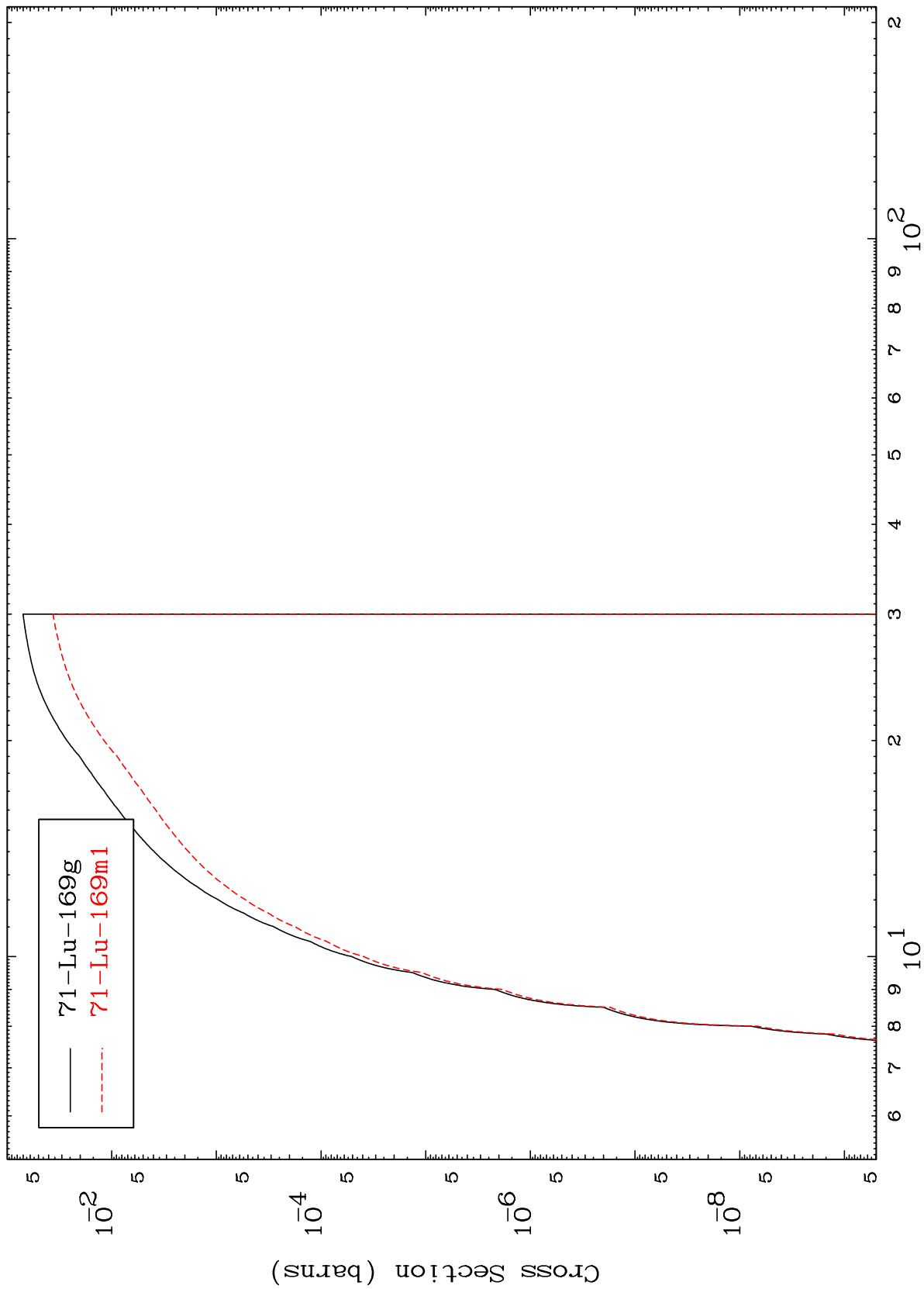
72-Hf-170

MAT 7213

(n,n') p

72-Hf-170

Radionuclide Production Cross Section



76

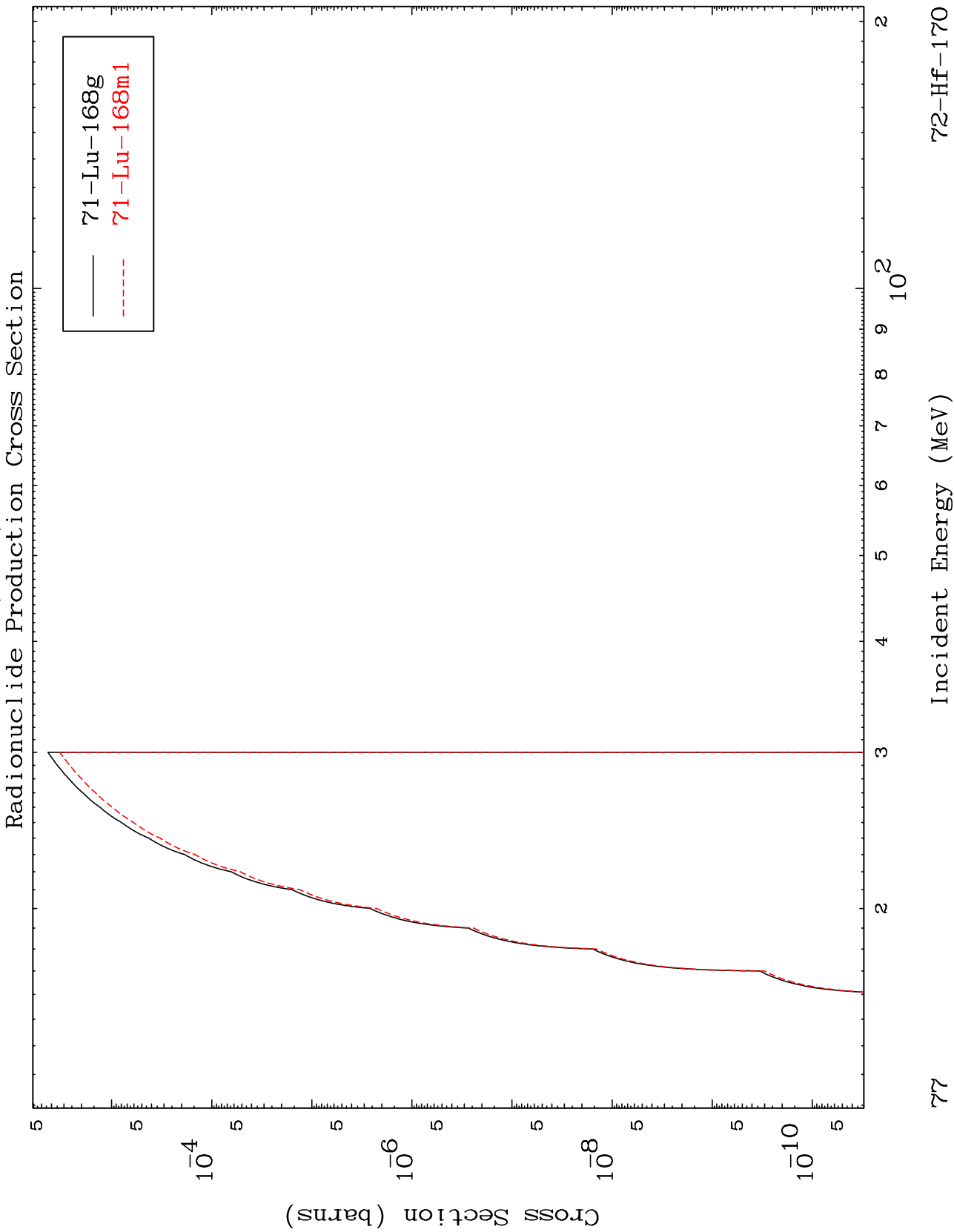
Incident Energy (MeV)

72-Hf-170

MAT 7213

(n,n') d

72-Hf-170



77

Incident Energy (MeV)

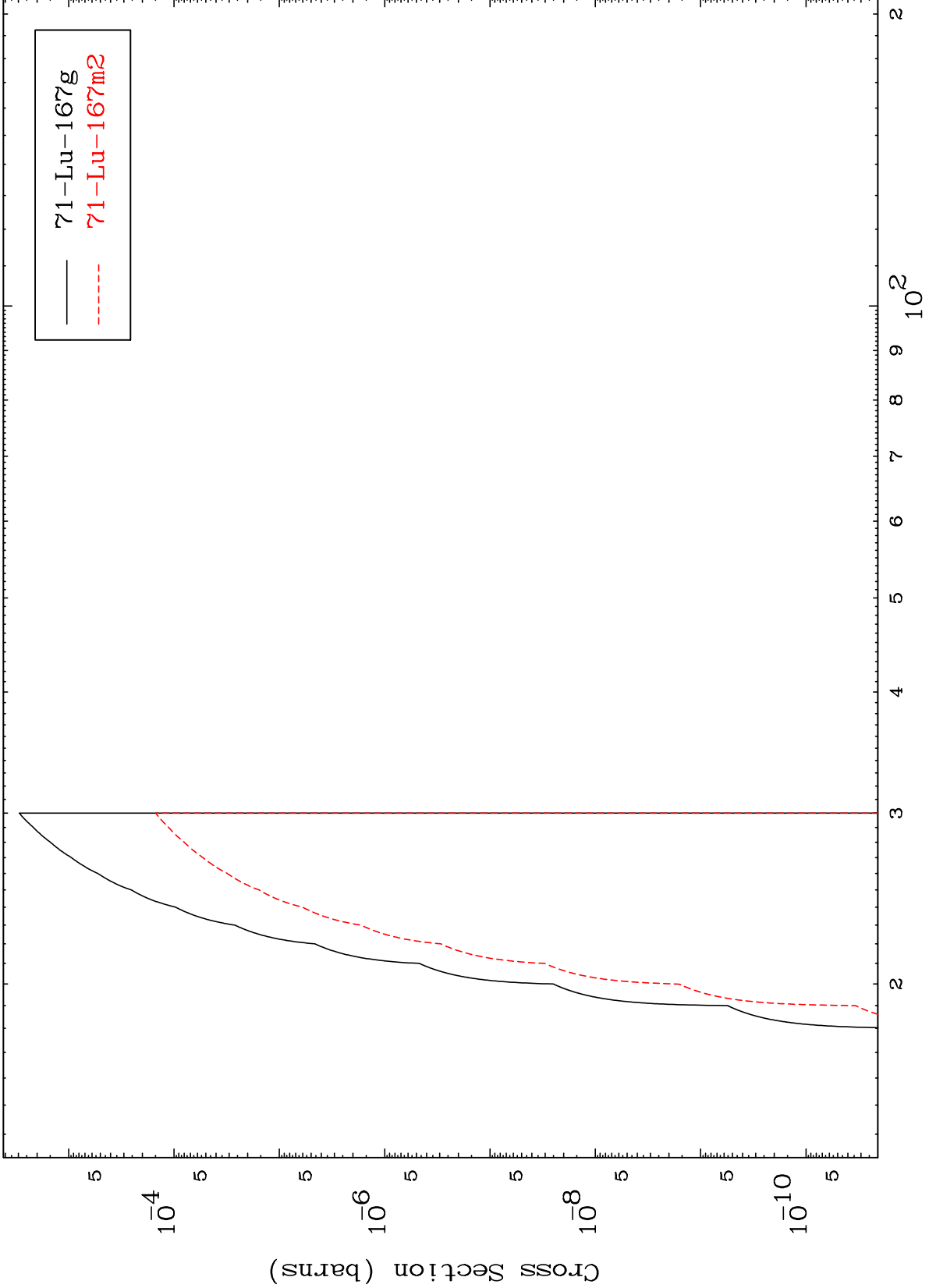
72-Hf-170

MAT 7213

(n,n') t

72-Hf-170

Radionuclide Production Cross Section



78

Incident Energy (MeV)

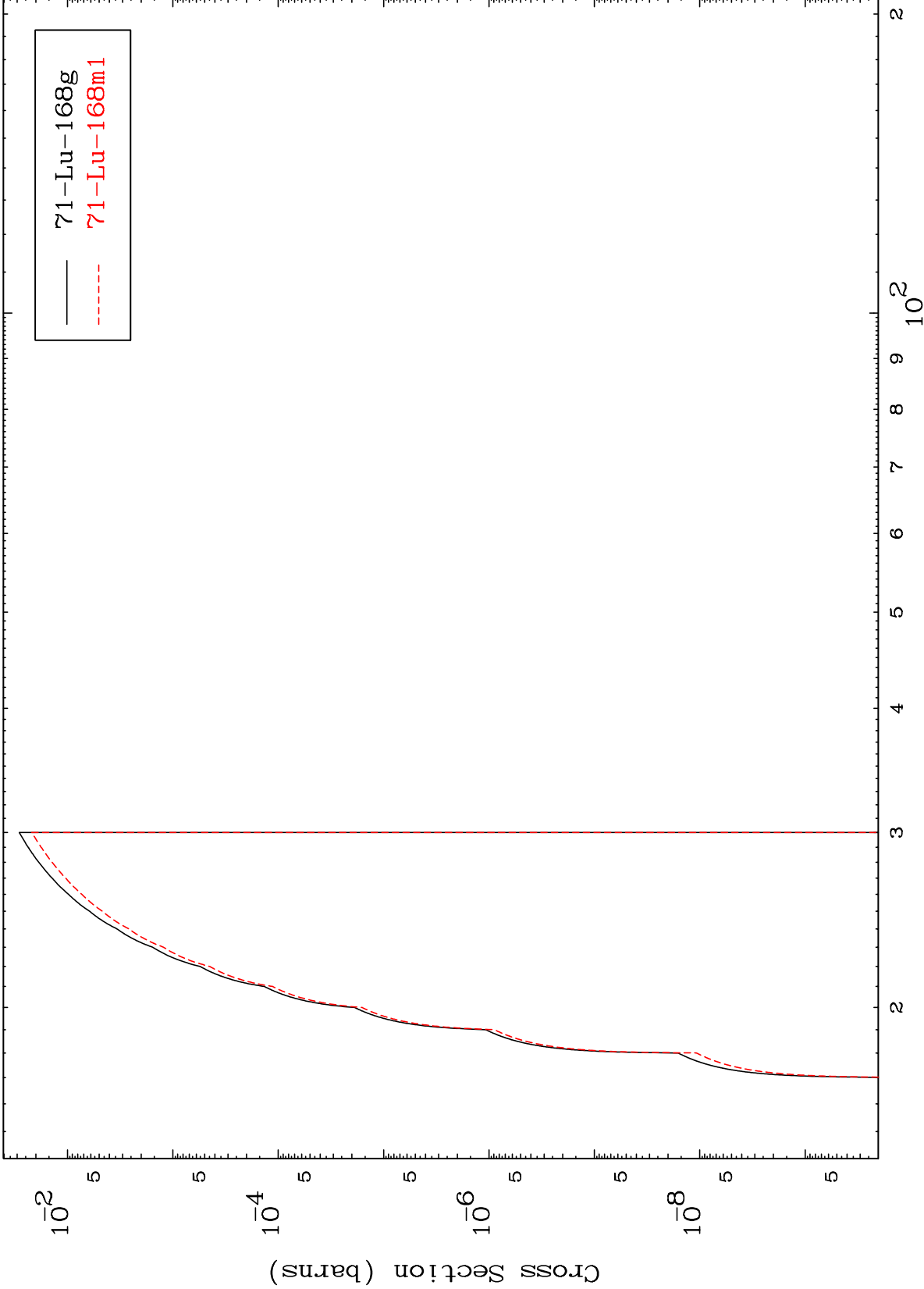
72-Hf-170

MAT 7213

(n,2n) p

72-Hf-170

Radionuclide Production Cross Section



79

Incident Energy (MeV)

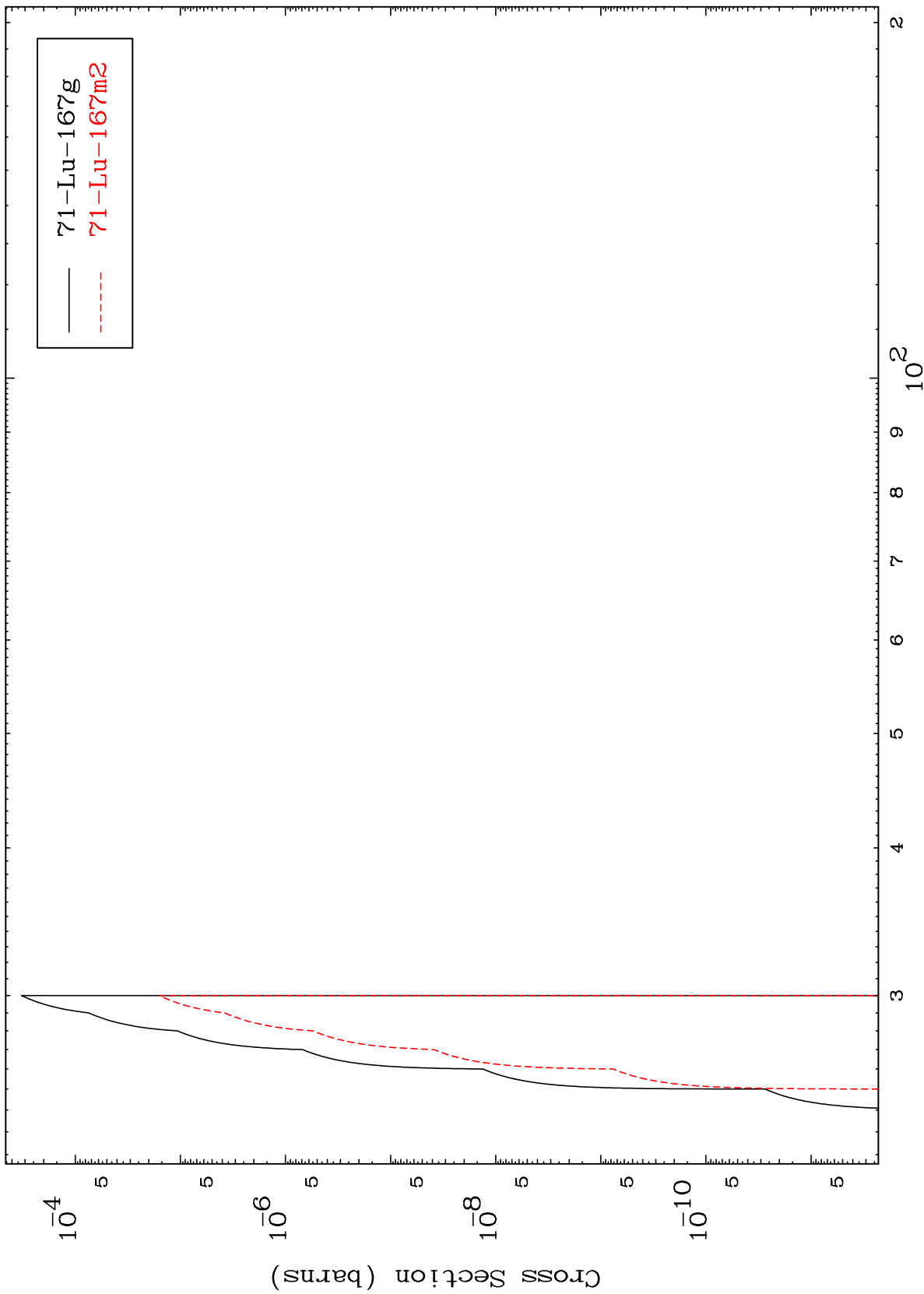
72-Hf-170

MAT 7213

(n,3n) p

72-Hf-170

Radionuclide Production Cross Section



80

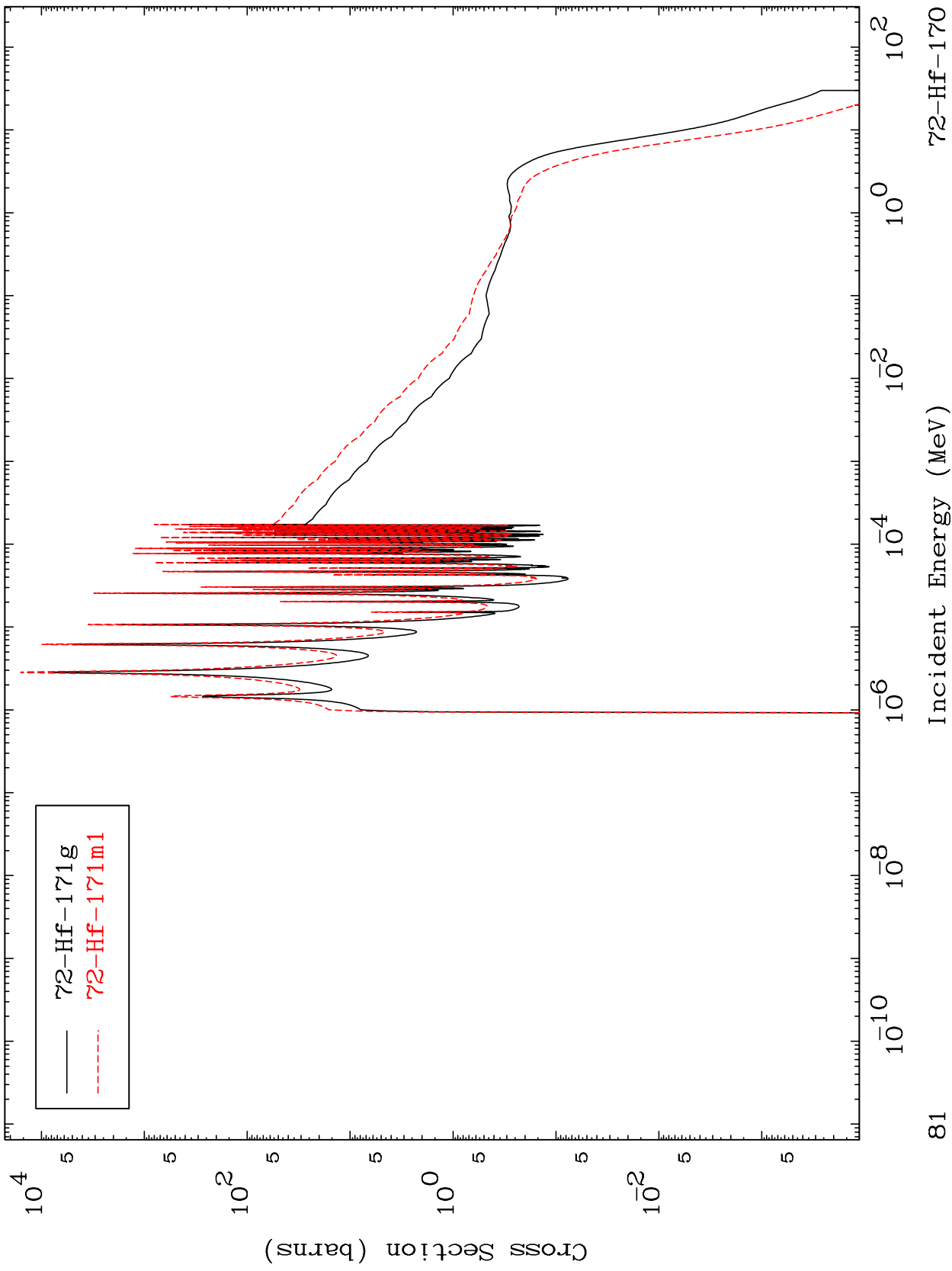
Incident Energy (MeV)

72-Hf-170

MAT 7213

⁷²Hf-170

(n,γ)
Radionuclide Production Cross Section

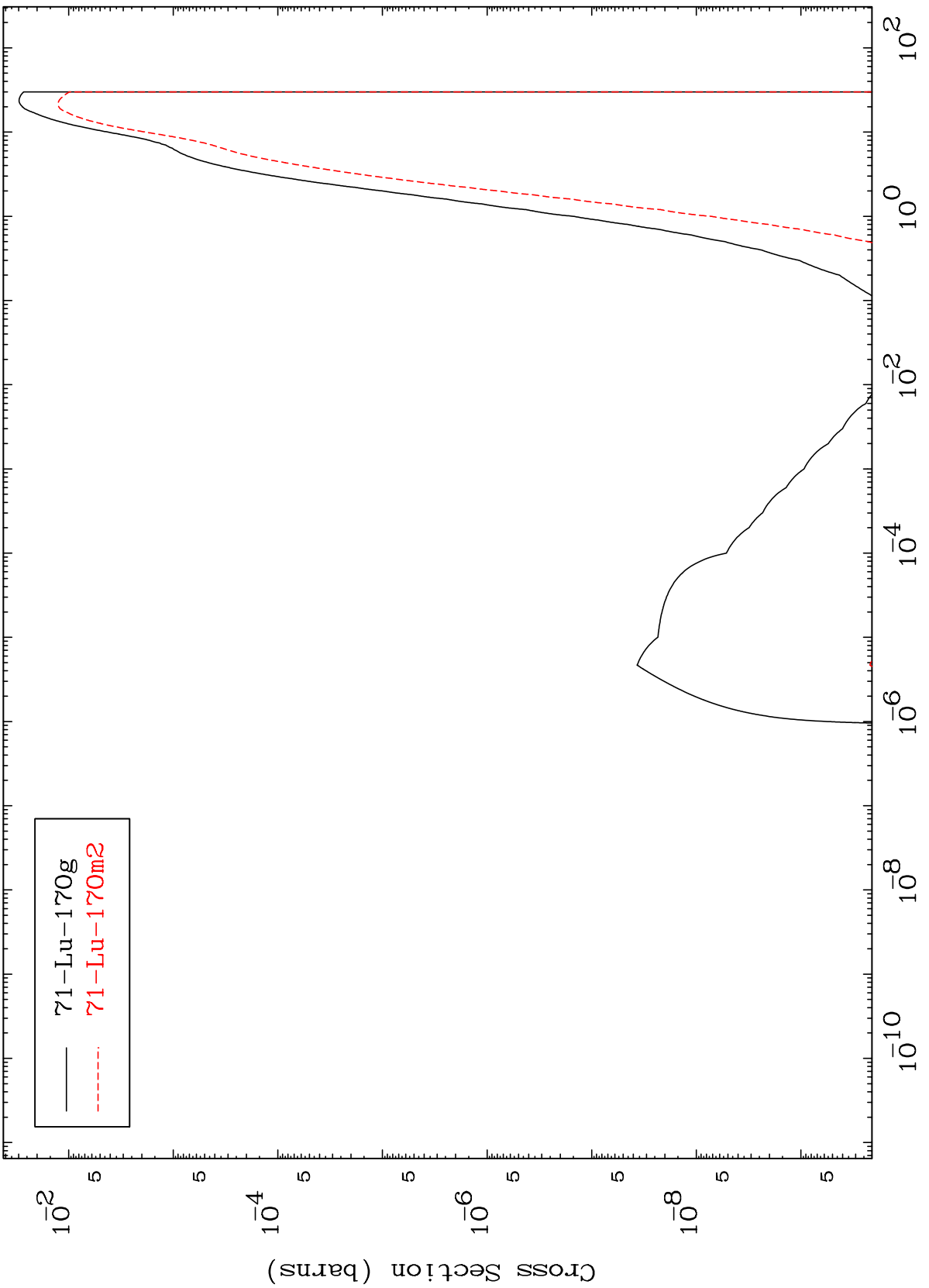
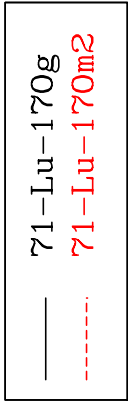


MAT 7213

(n,p)

72-Hf-170

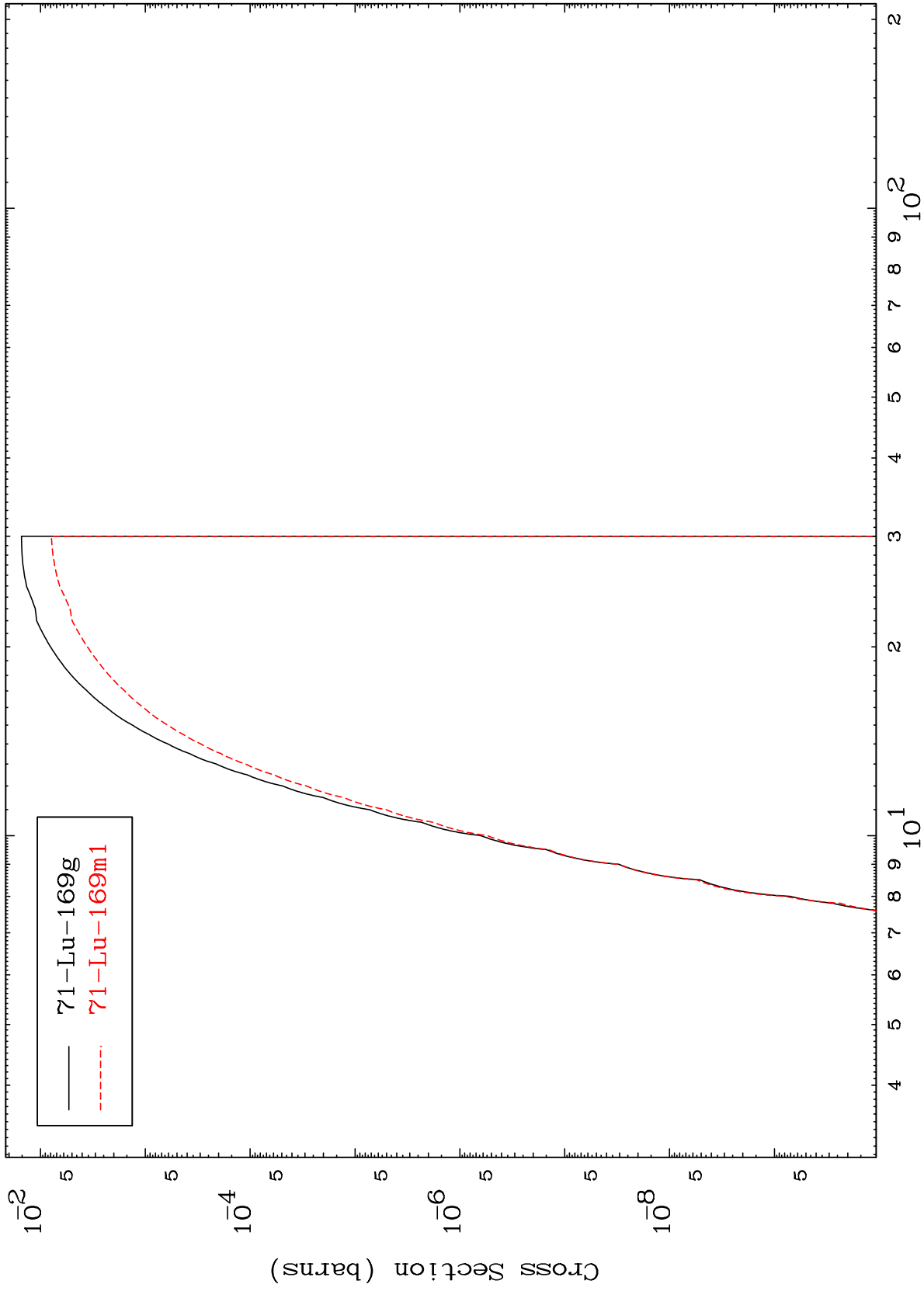
Radionuclide Production Cross Section



MAT 7213

72-Hf-170

(n,d)
Radionuclide Production Cross Section



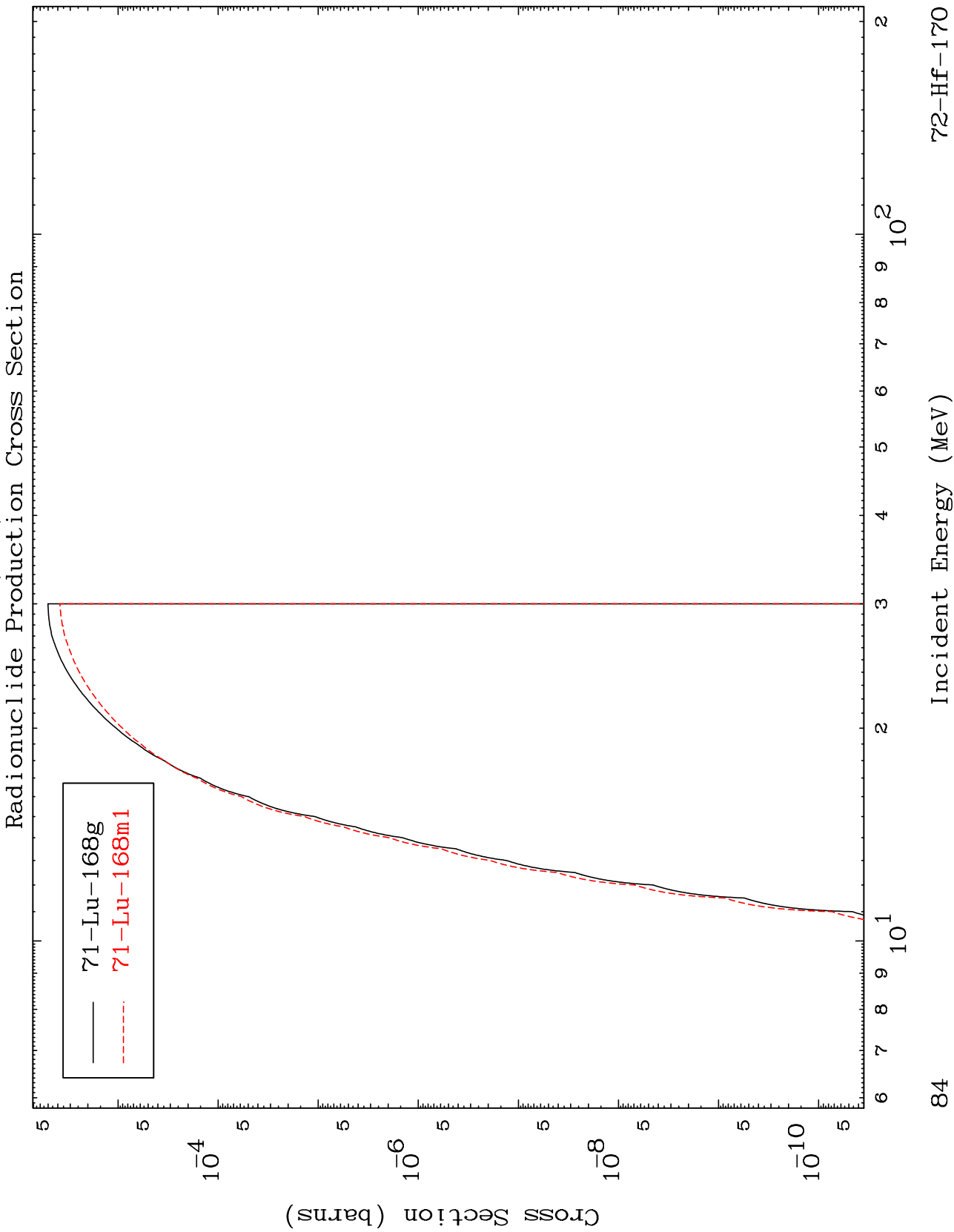
83

Incident Energy (MeV)

72-Hf-170

MAT 7213

72-Hf-170



Incident Energy (MeV)

72-Hf-170

84

MAT 7213

72-Hf-170

(n,2p)
Radionuclide Production Cross Section

