

Program EVALPLOT
(Version 2021-1)

by

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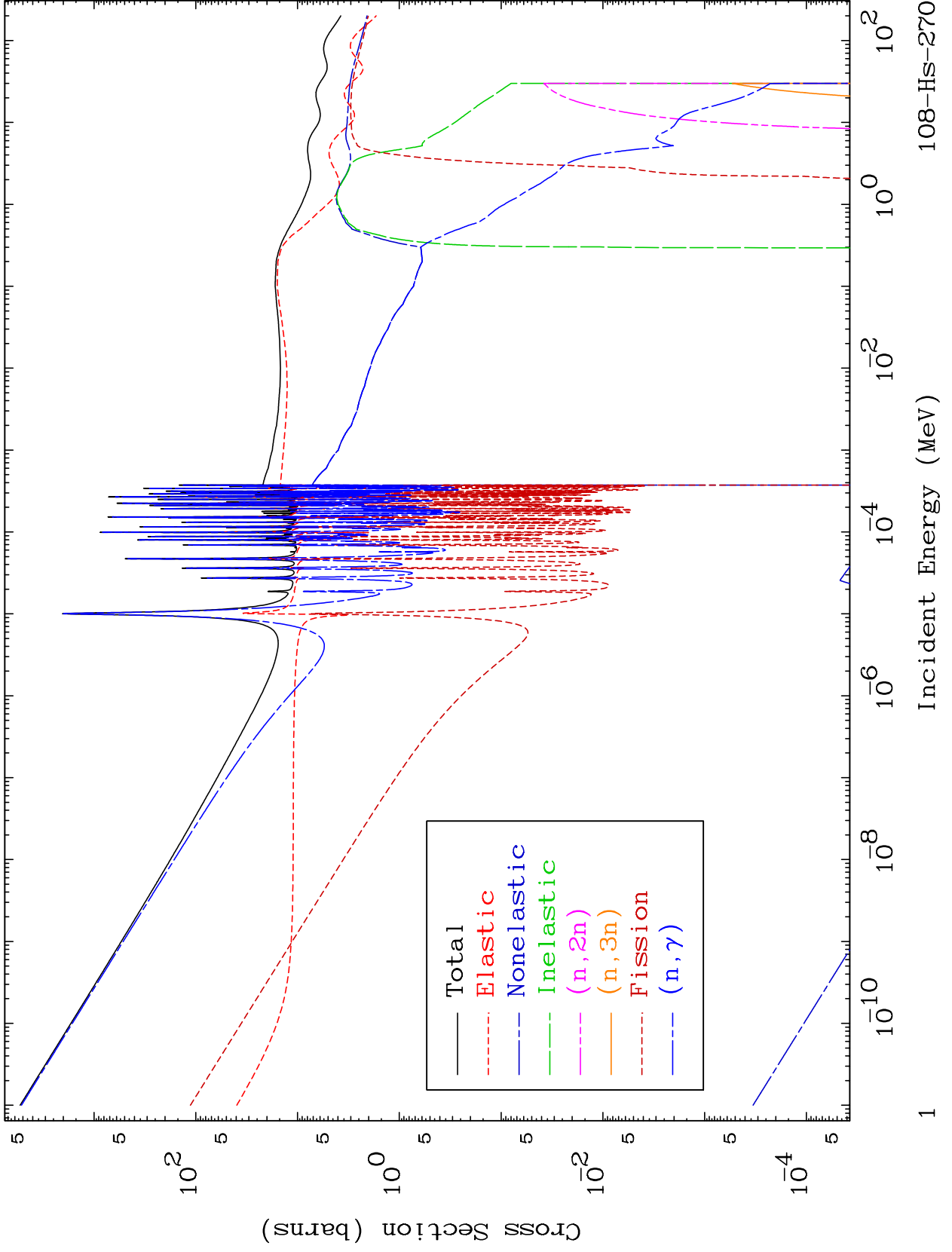
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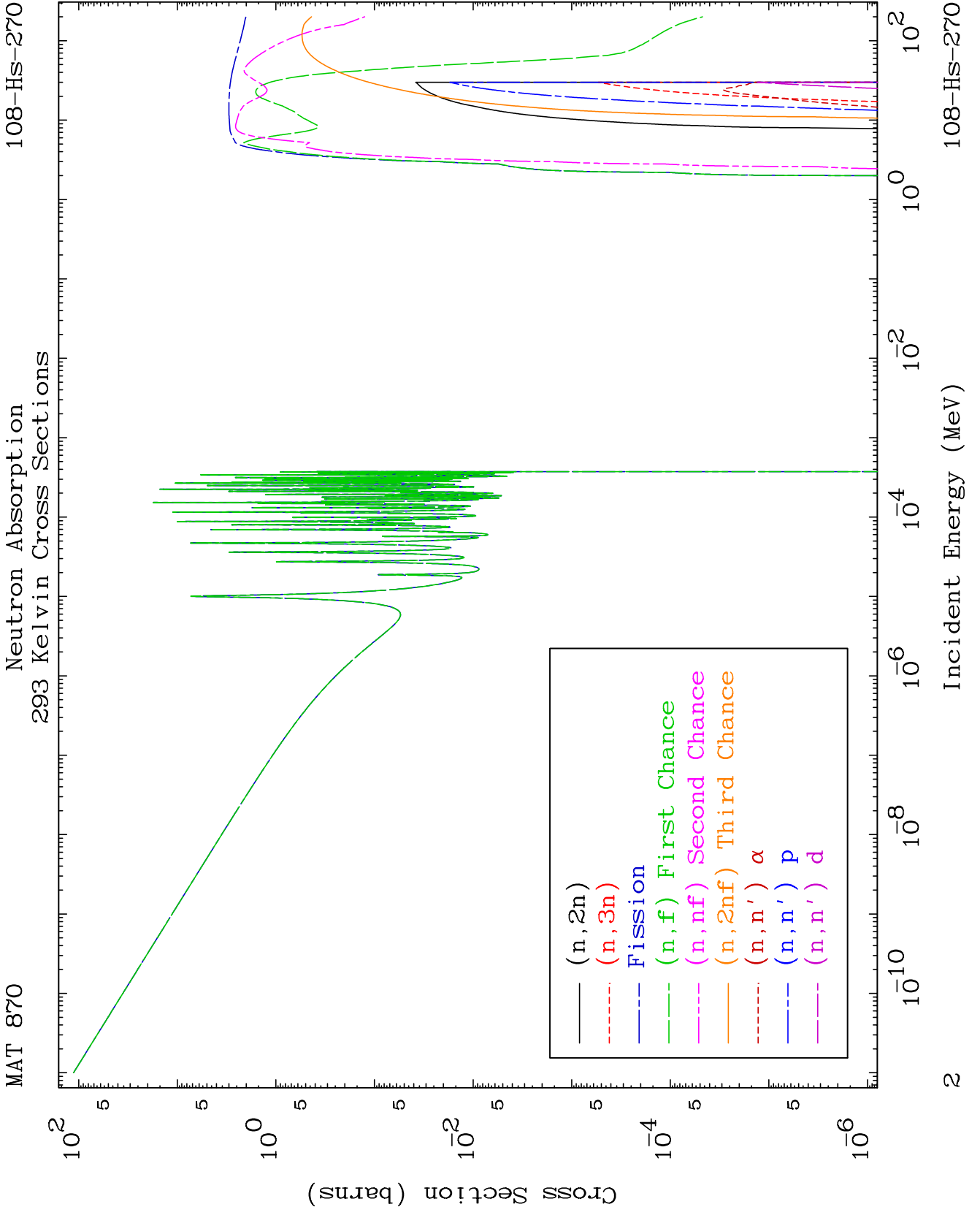
Press Mouse Button to Start

MAT 870

Neutron Major
293 Kelvin Cross Sections

108-Hs-270

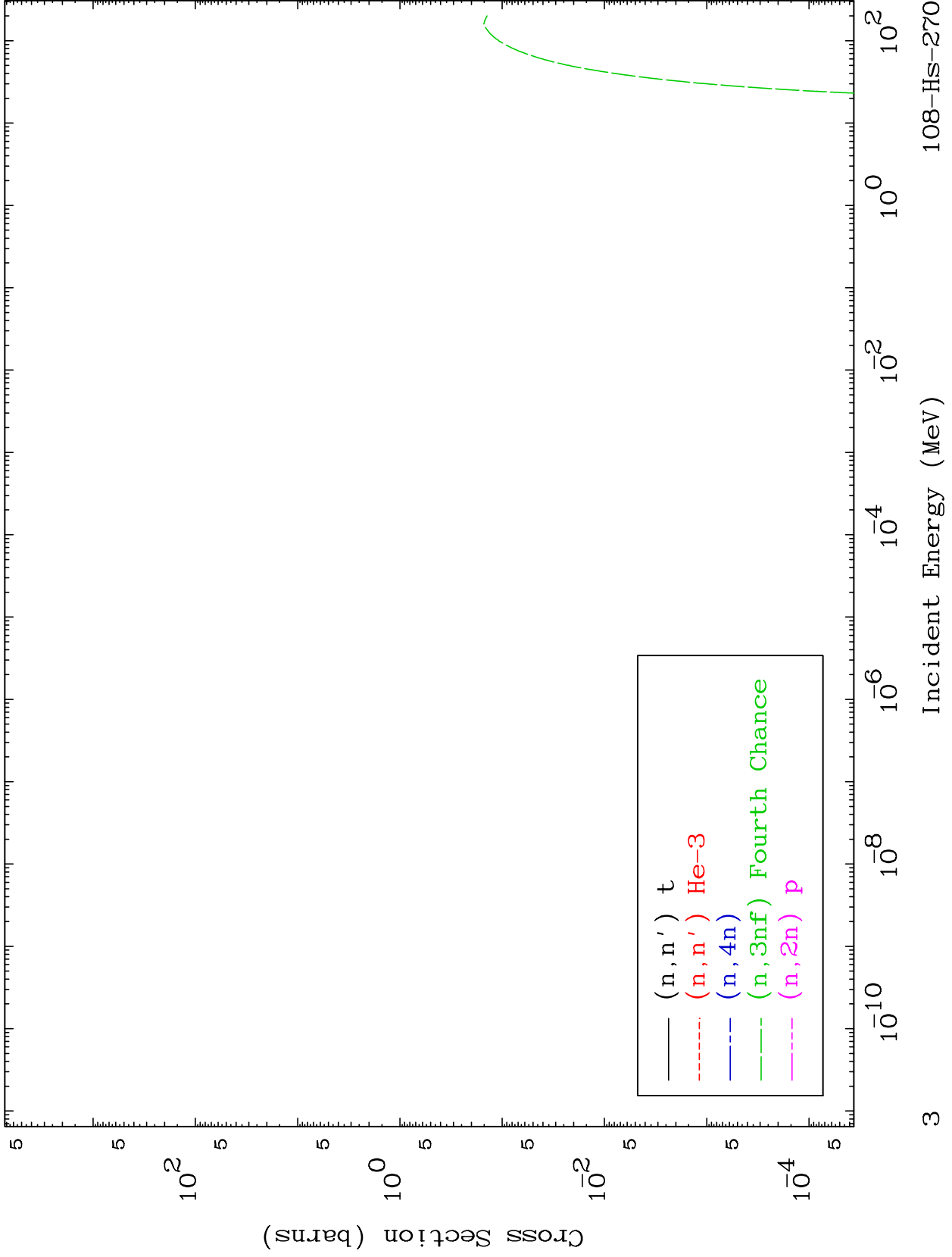




MAT 870

Neutron Absorption
293 Kelvin Cross Sections

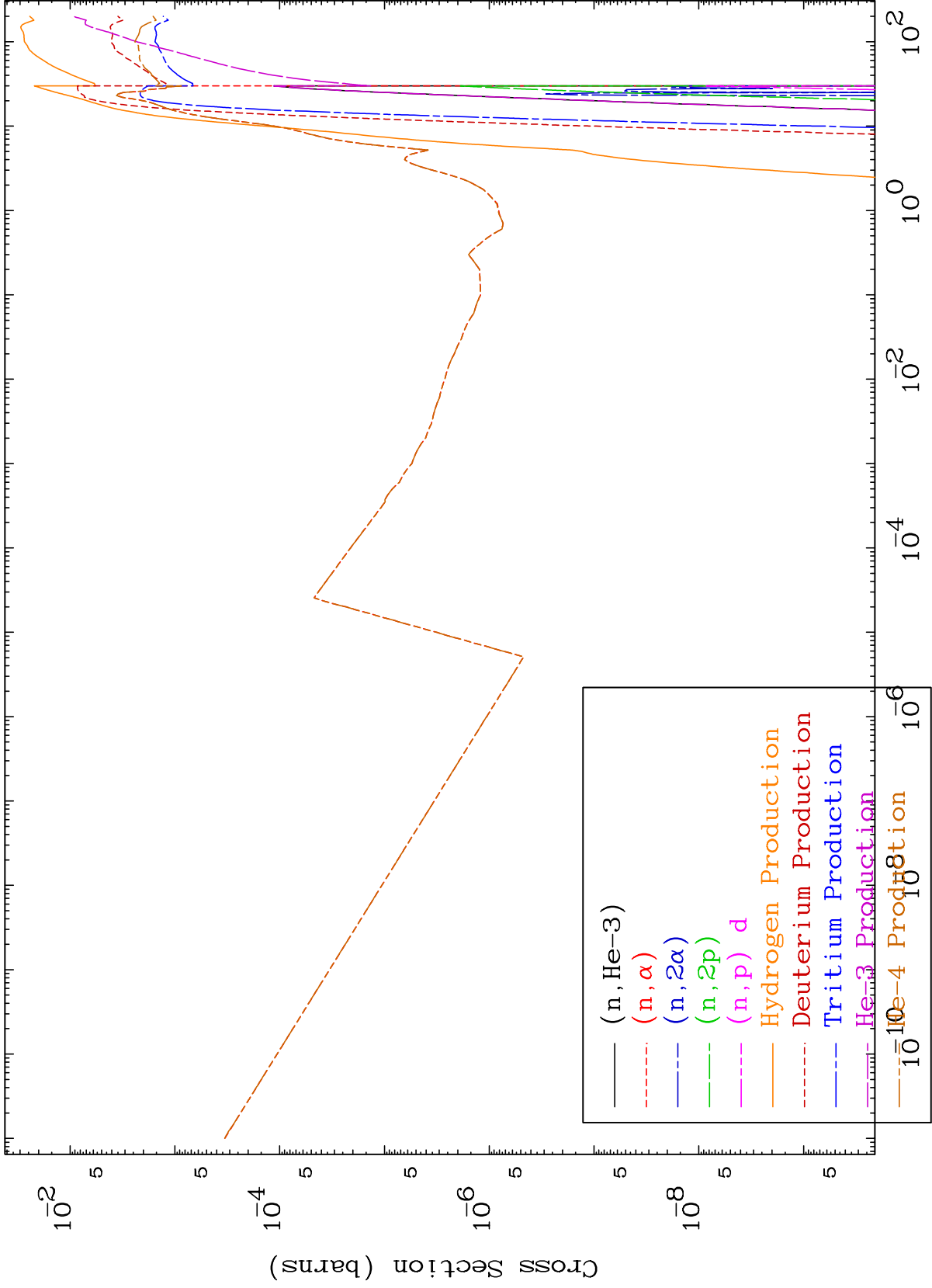
108-Hs-270



MAT 870

Neutron Absorption
293 Kelvin Cross Sections

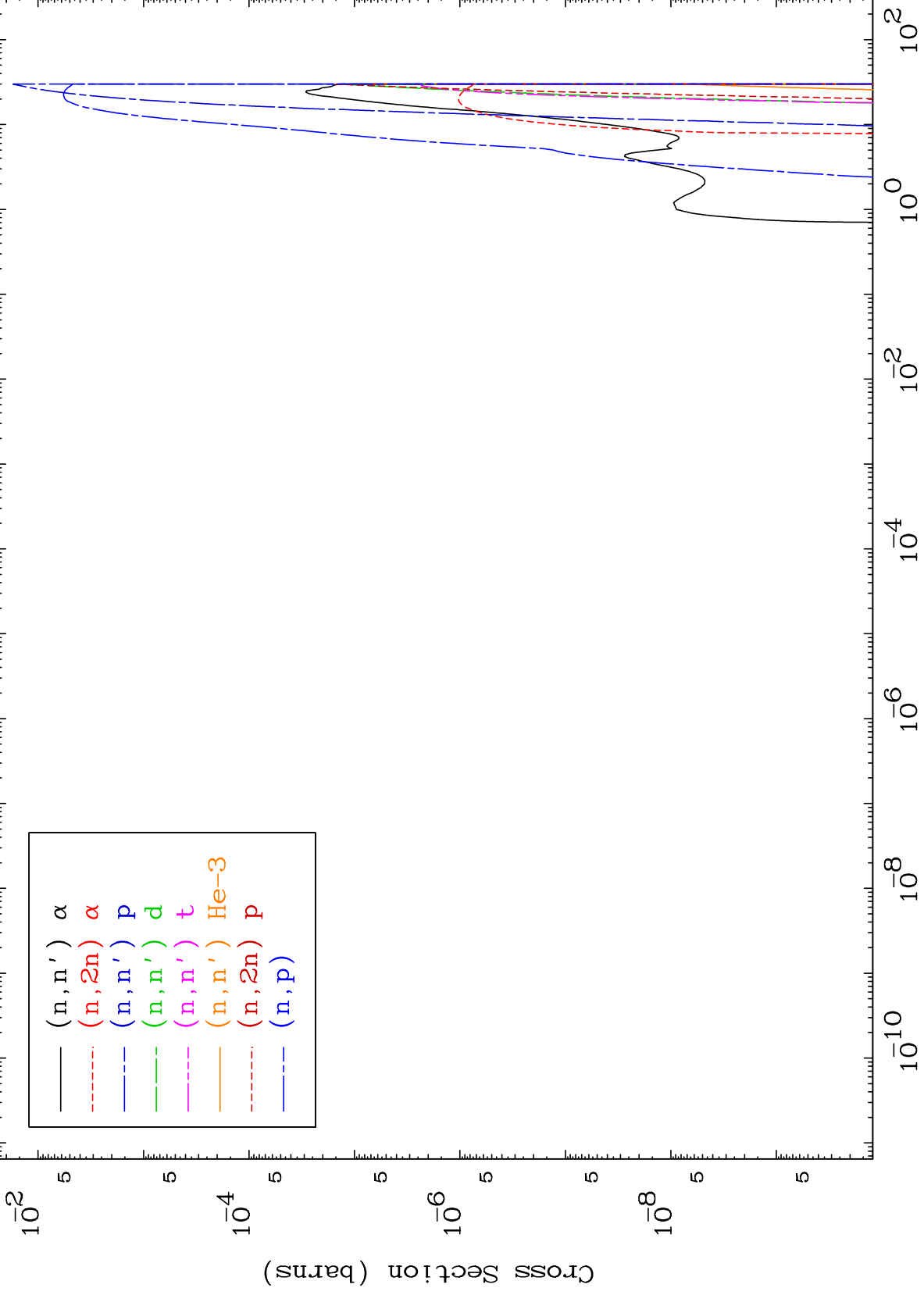
108-Hs-270

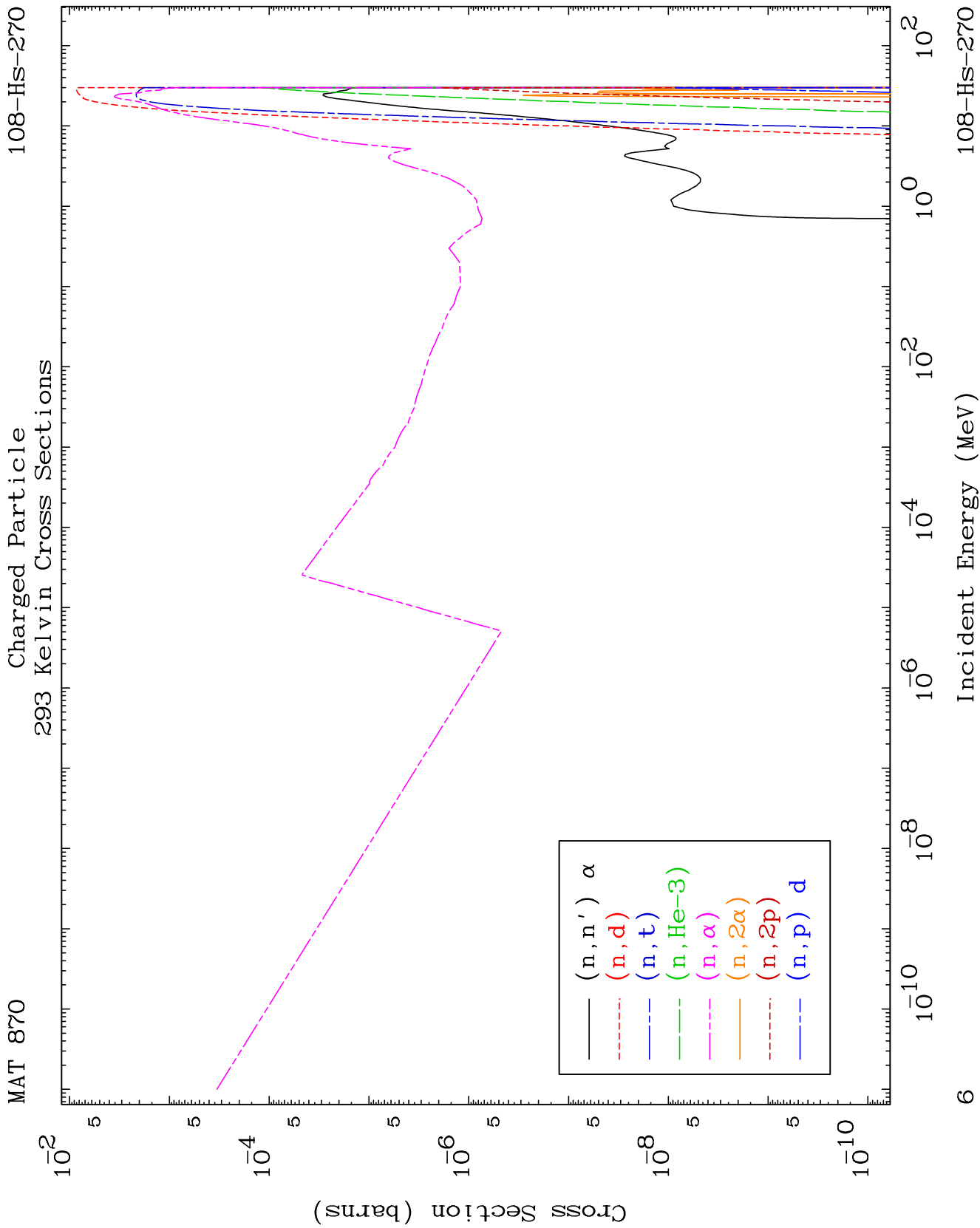


MAT 870

Charged Particle
293 Kelvin Cross Sections

108-Hs-270

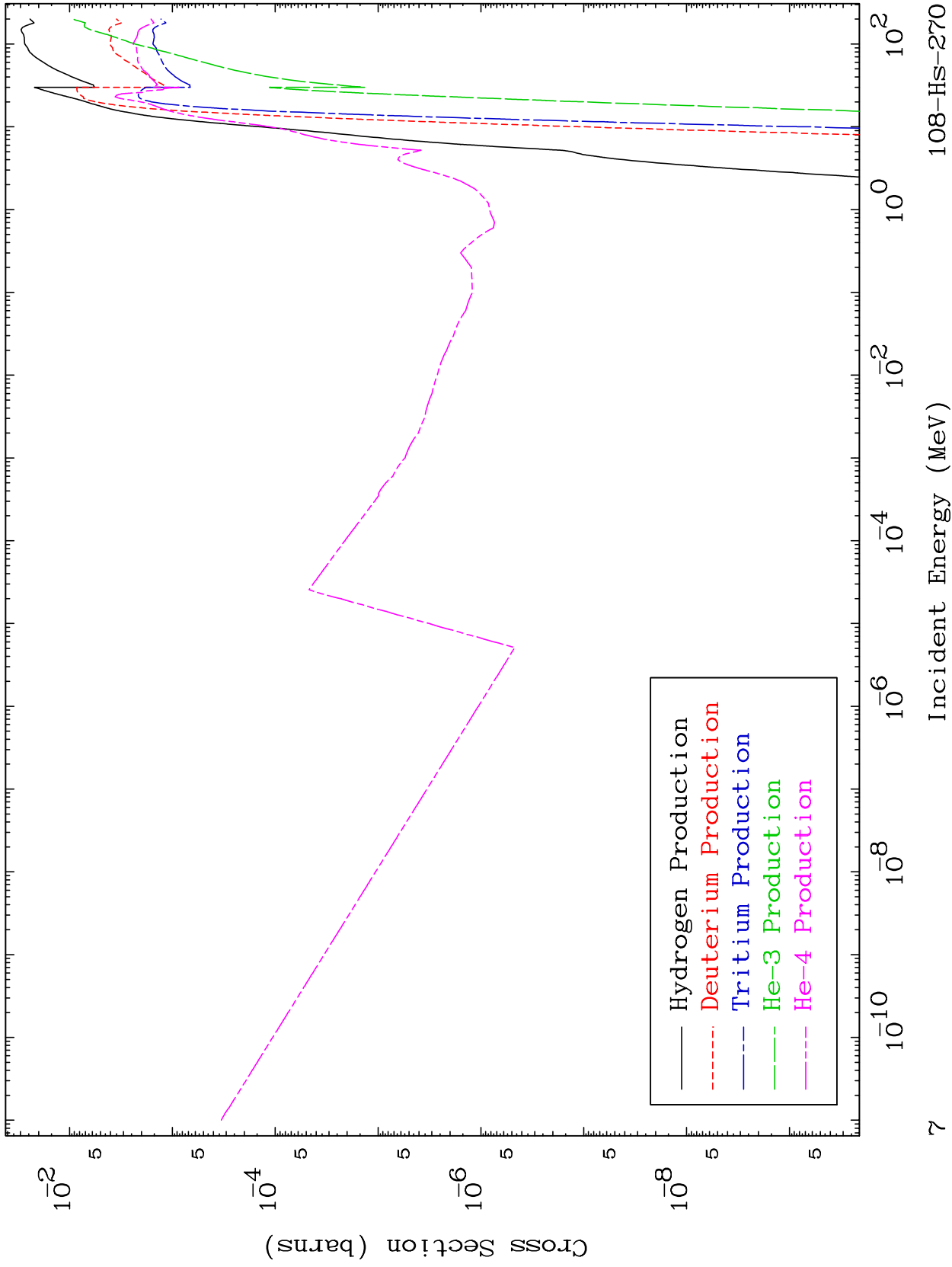


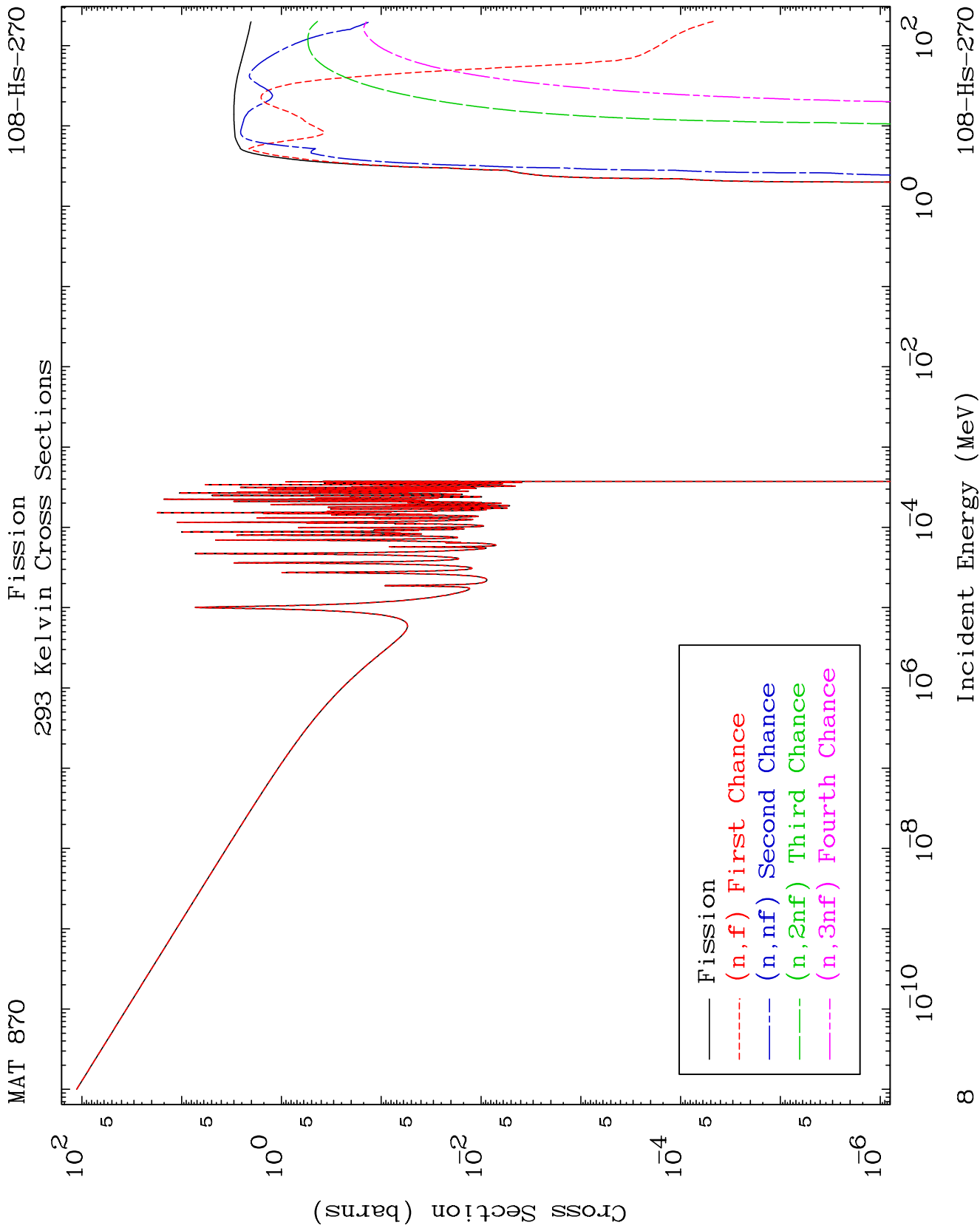


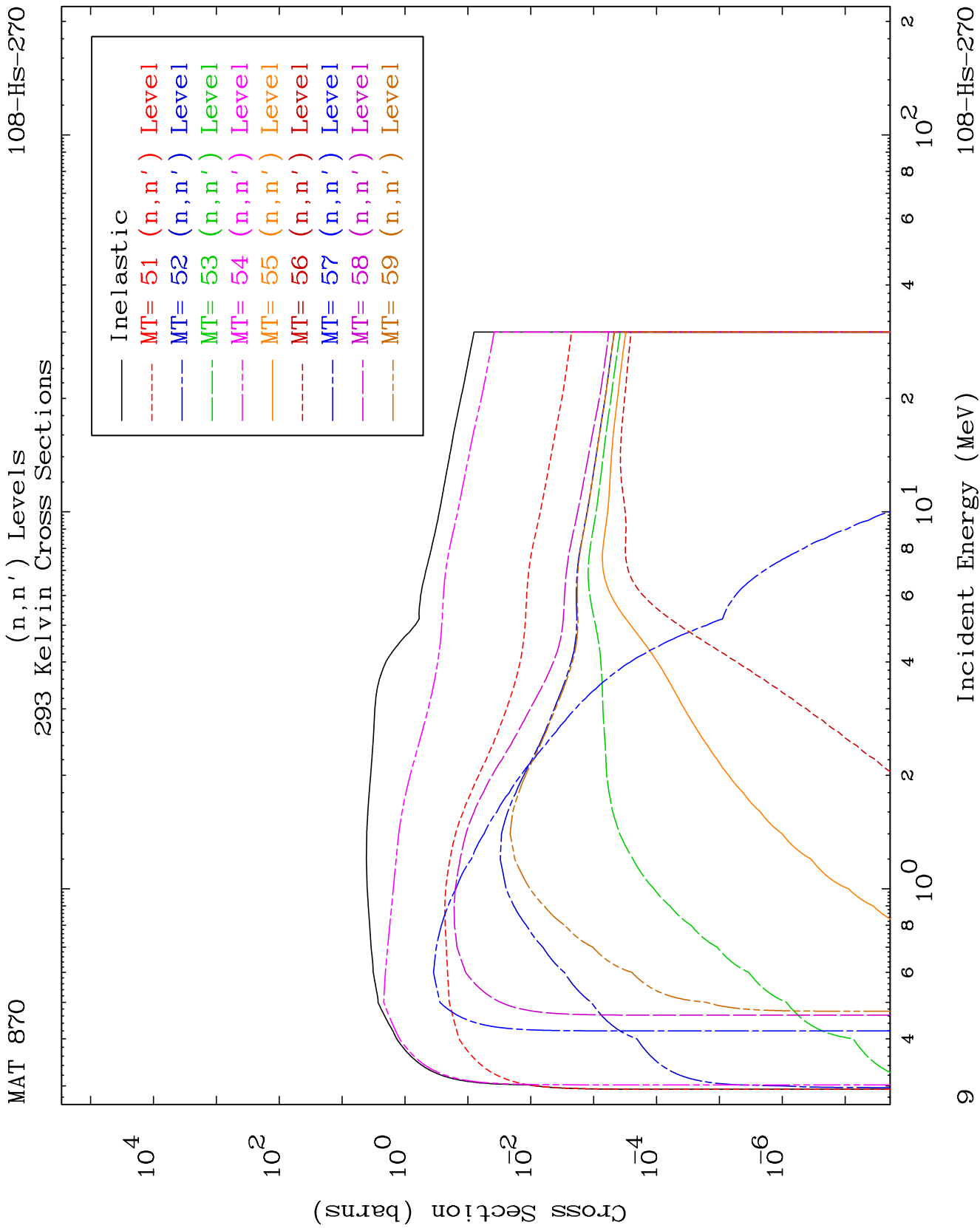
MAT 870

Particle Production
293 Kelvin Cross Sections

108-Hs-270



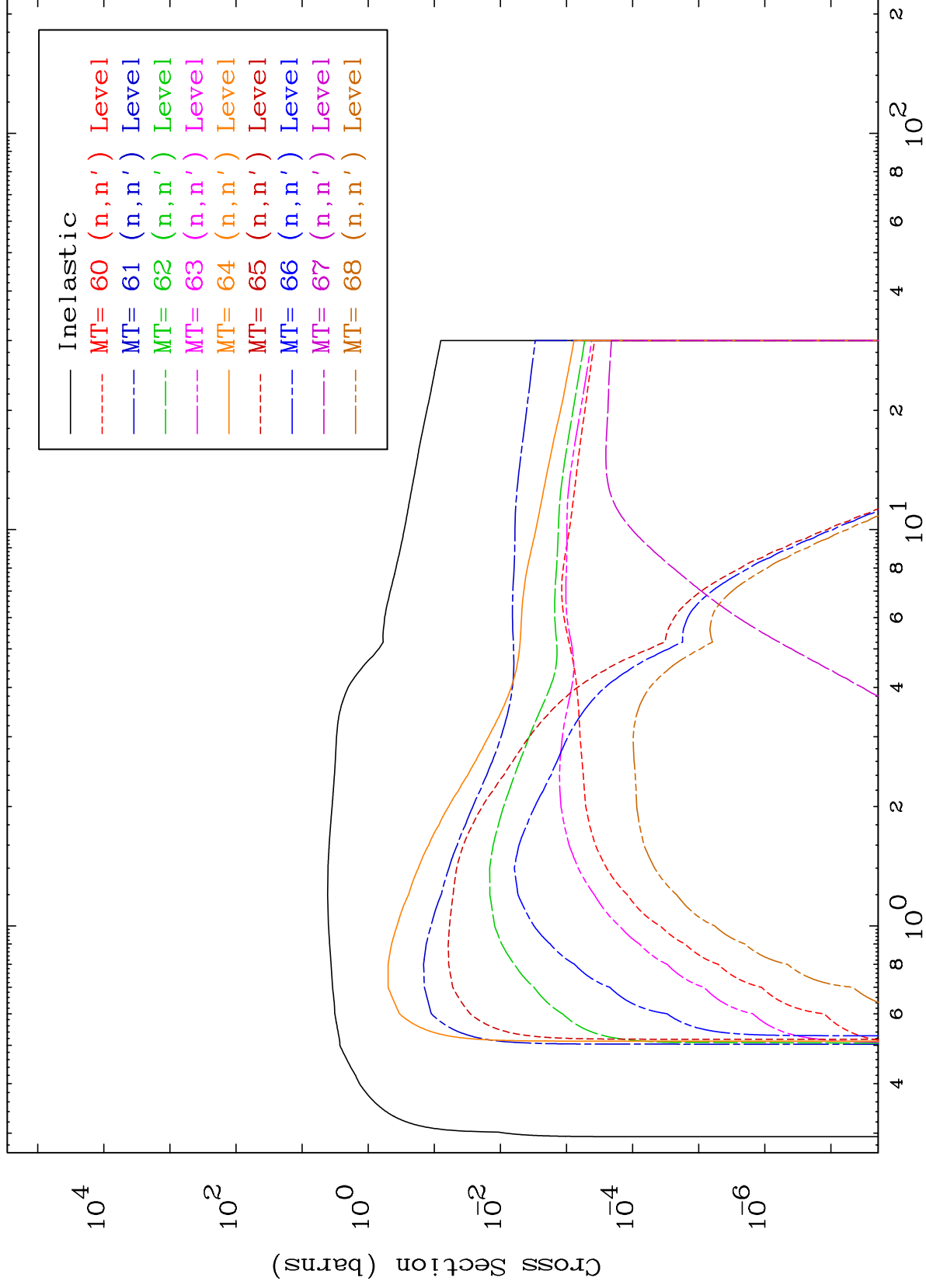




MAT 870

(n,n') Levels
293 Kelvin Cross Sections

108-Hs-270



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Incident Energy (MeV)

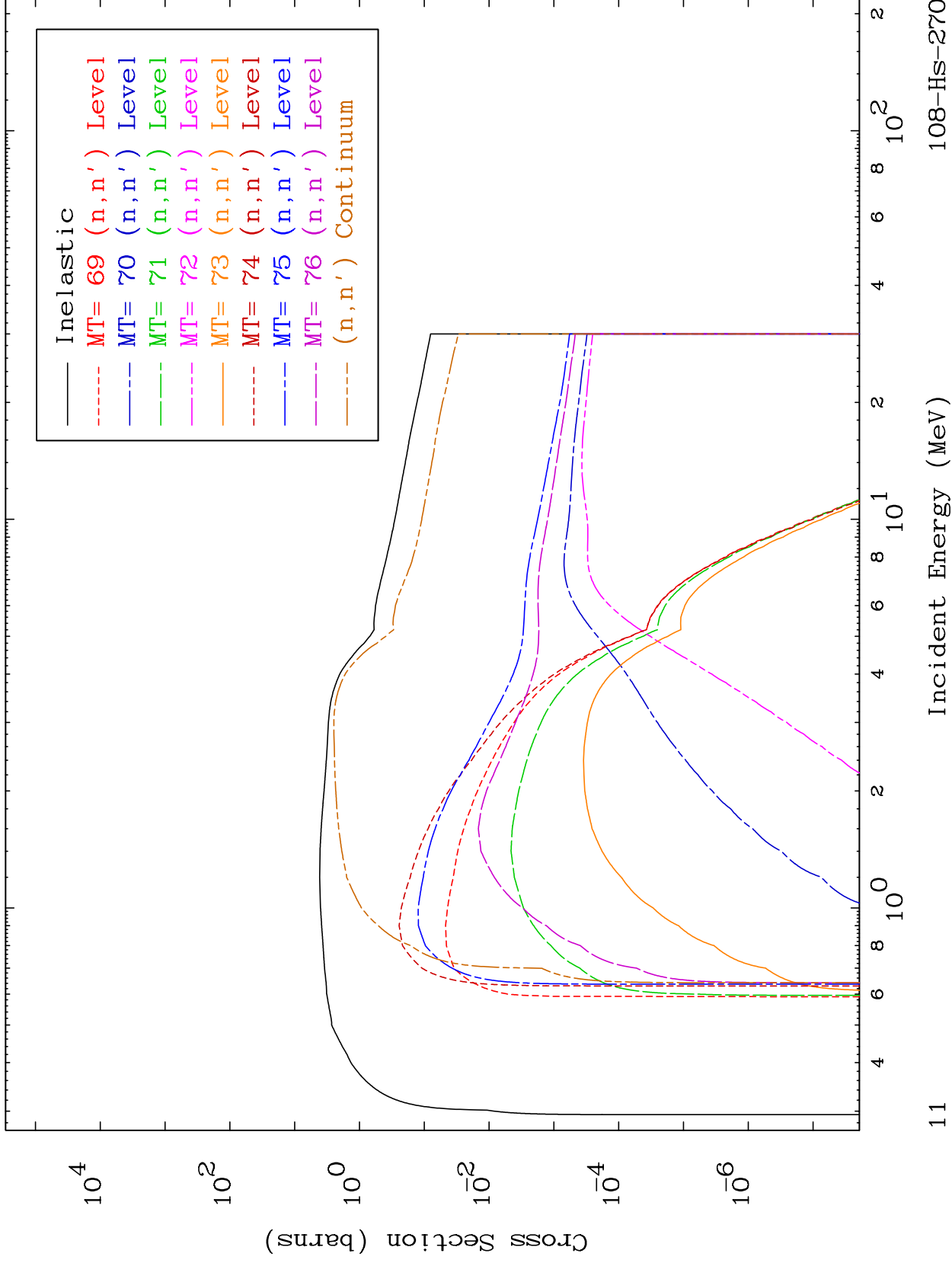
108-Hs-270

MAT 870

(n,n') Levels

293 Kelvin Cross Sections

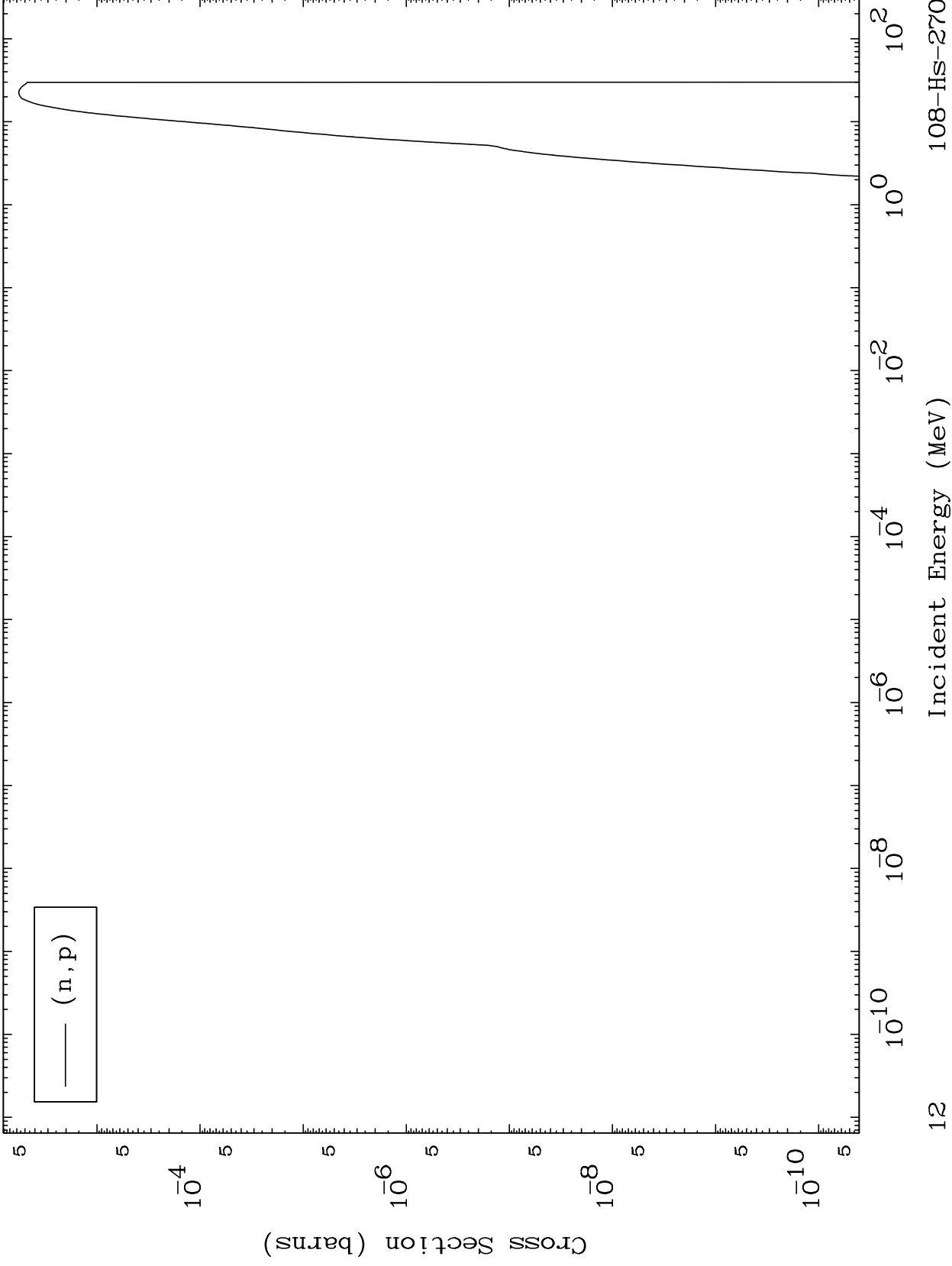
108-Hs-270

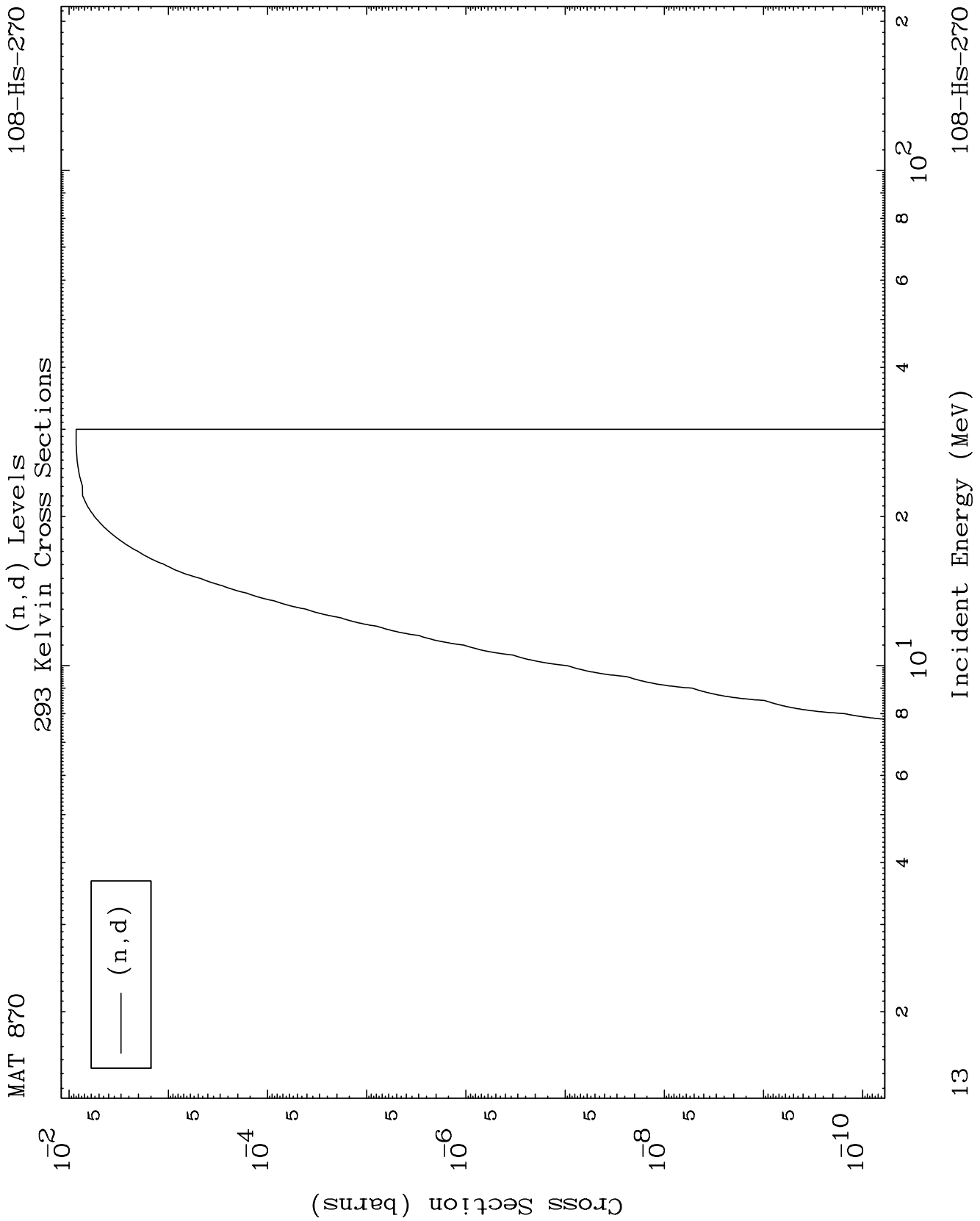


MAT 870

(n,p) Levels
293 Kelvin Cross Sections

108-Hs-270

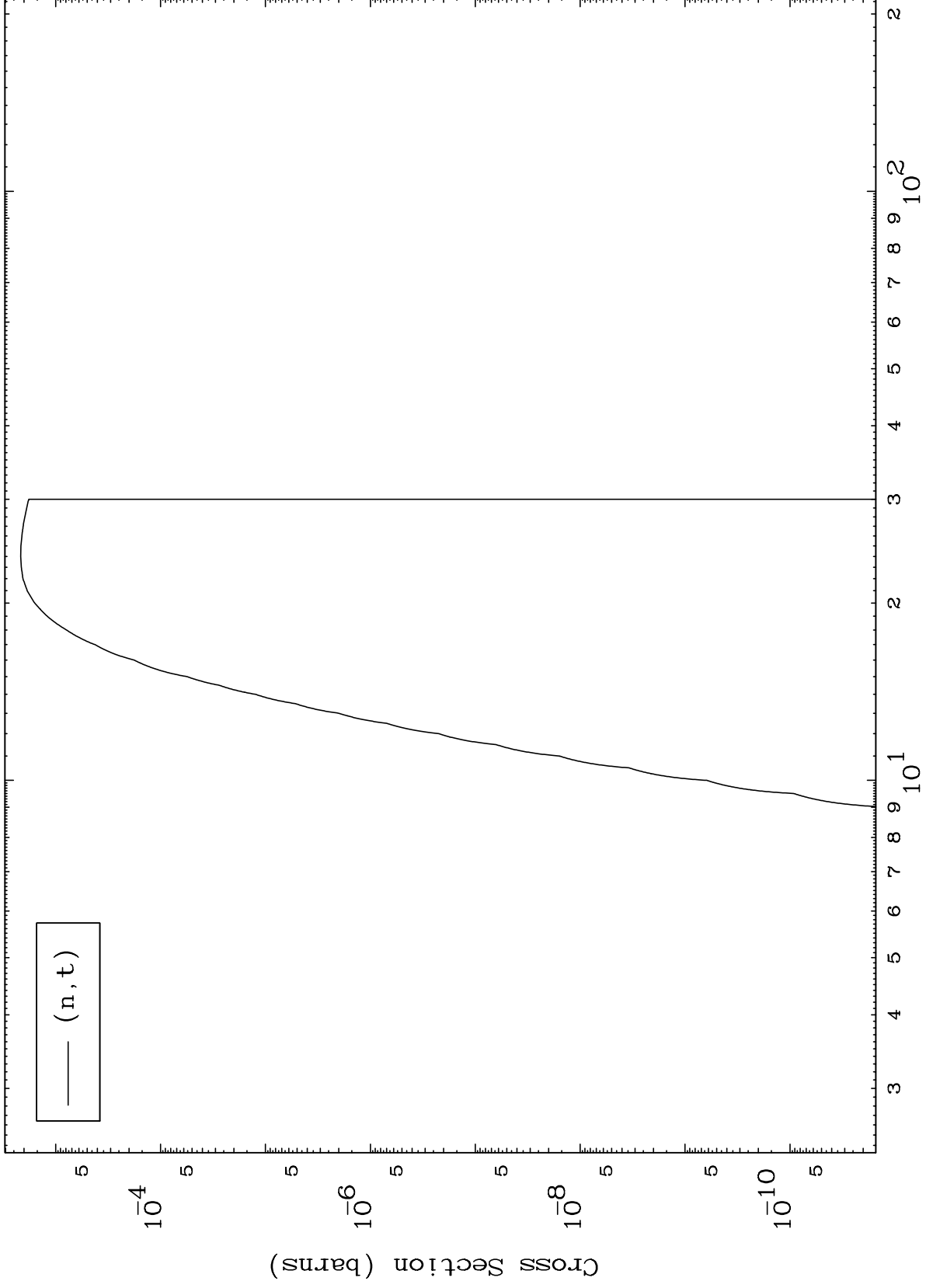




MAT 870

(n,t) Levels
293 Kelvin Cross Sections

108-Hs-270



(n,t)

Incident Energy (MeV)

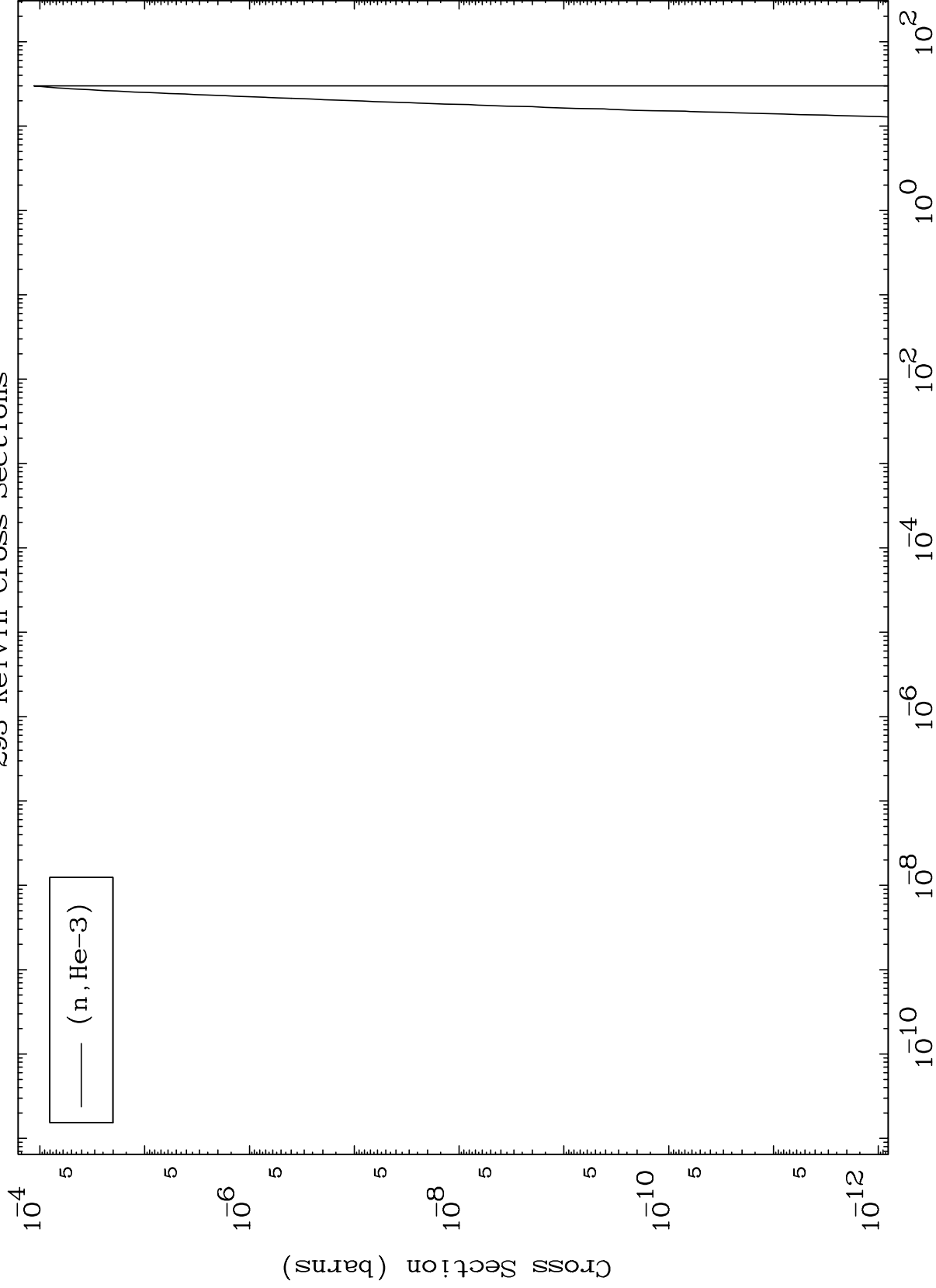
108-Hs-270

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MAT 870

(n,He3) Levels
293 Kelvin Cross Sections

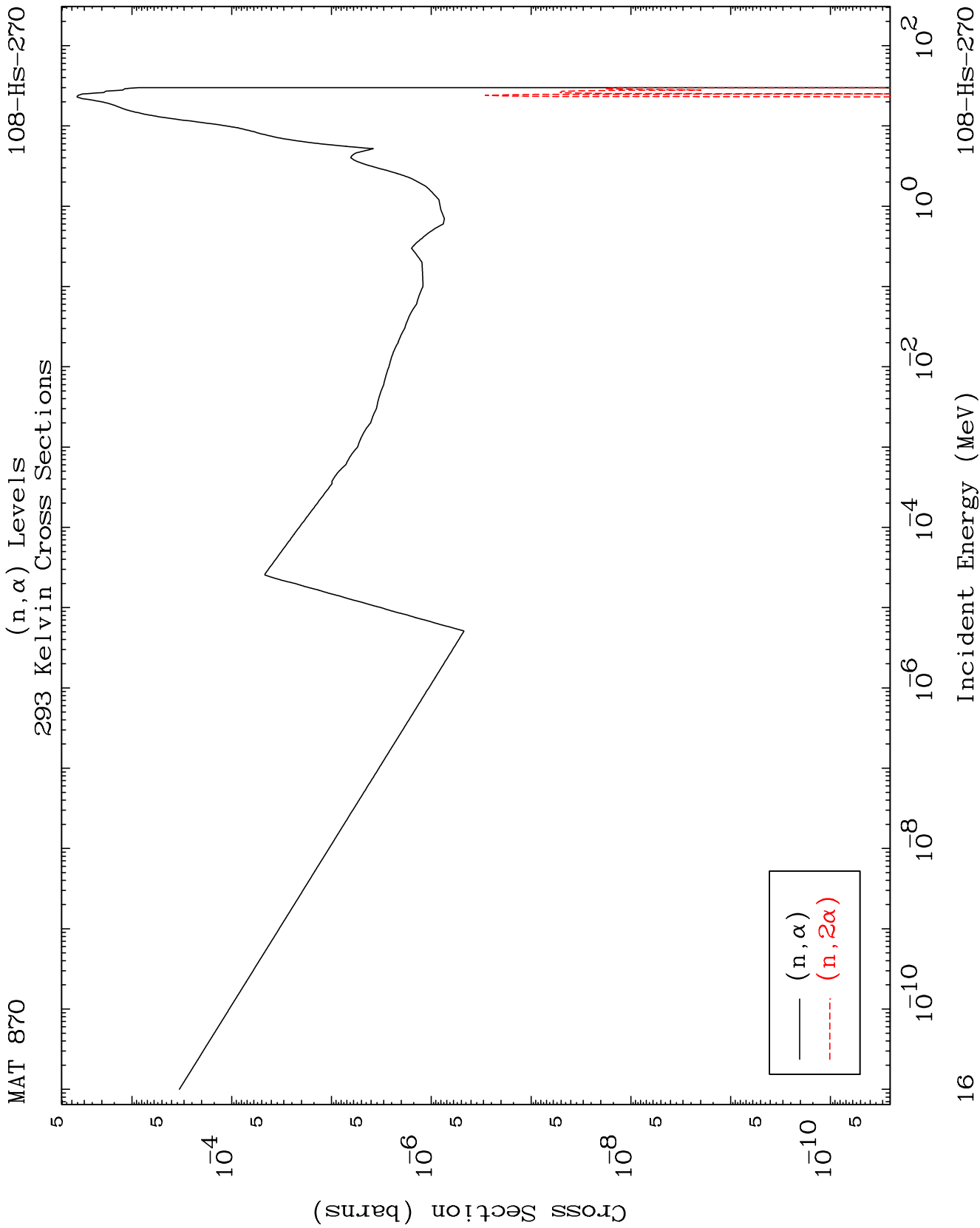
108-Hs-270



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Incident Energy (MeV)

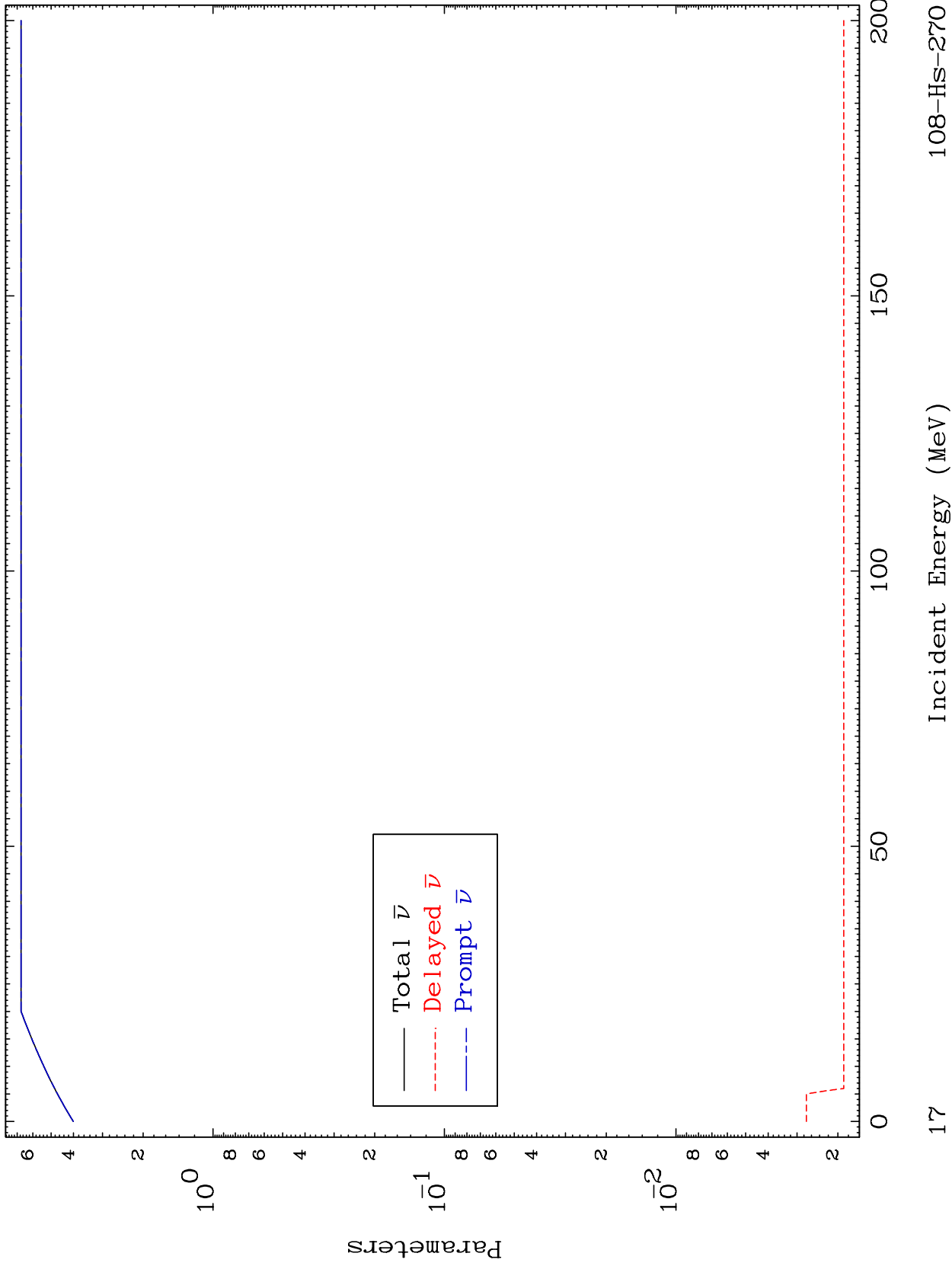
108-Hs-270



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Energy Release
Parameters

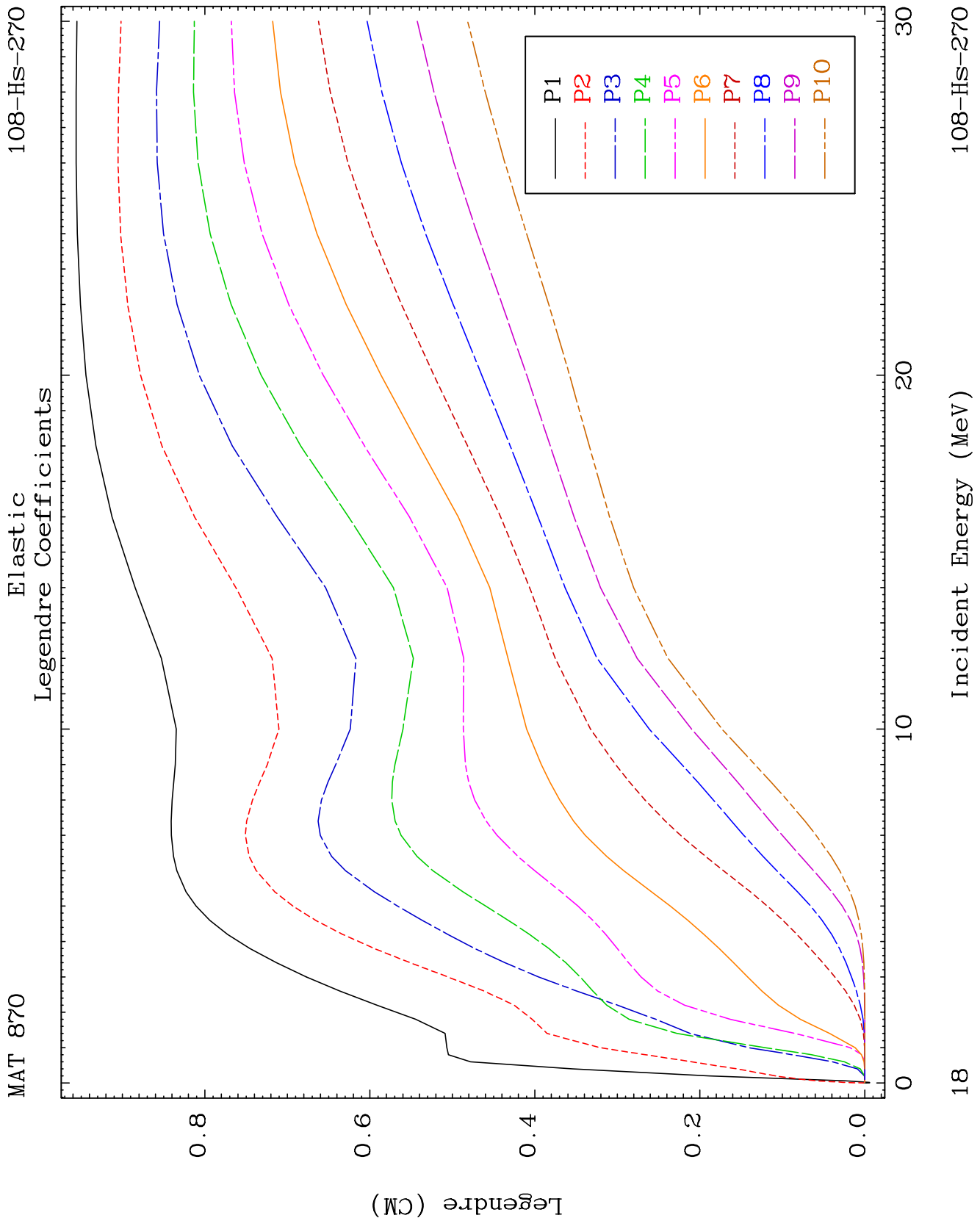
108-Hs-270

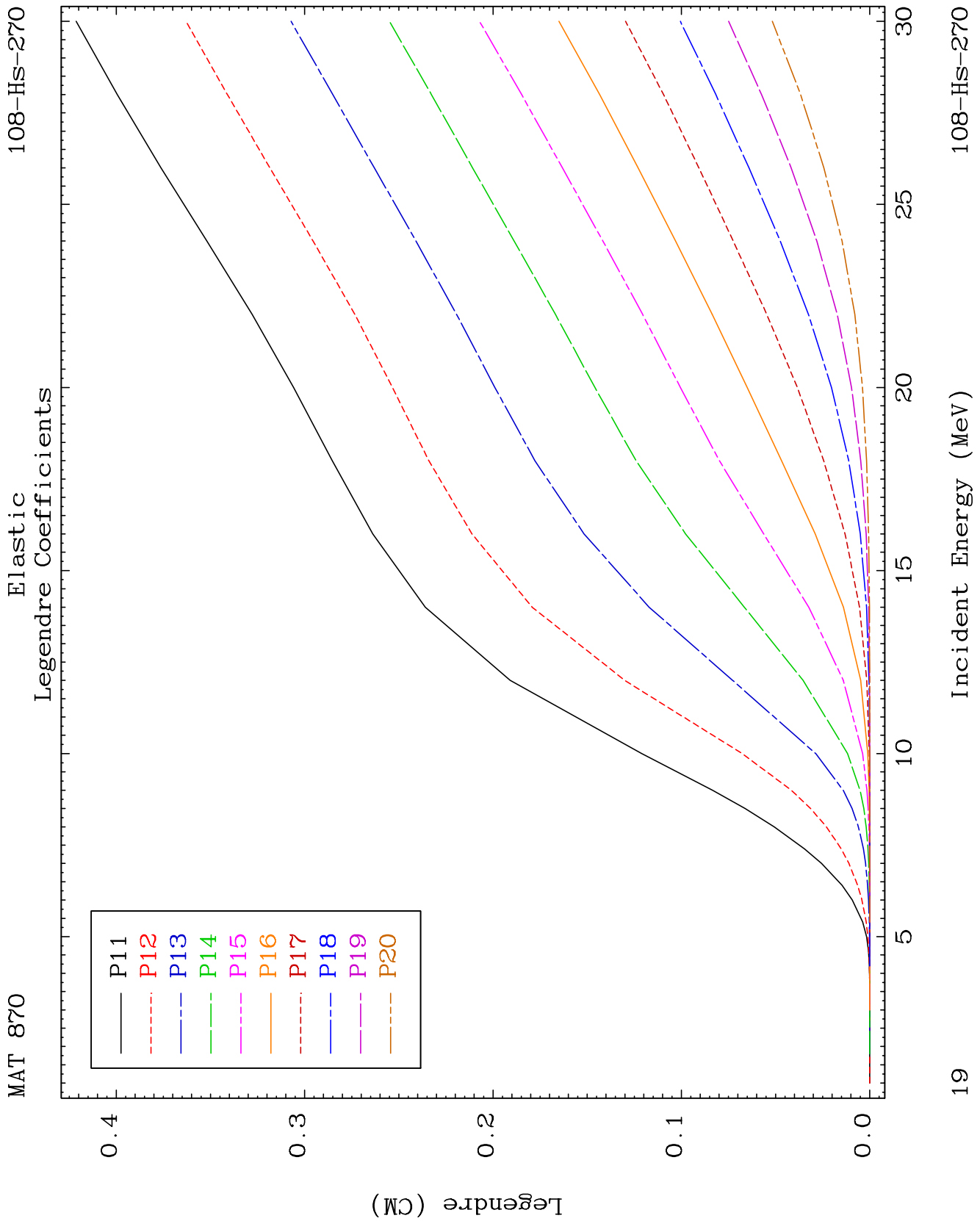


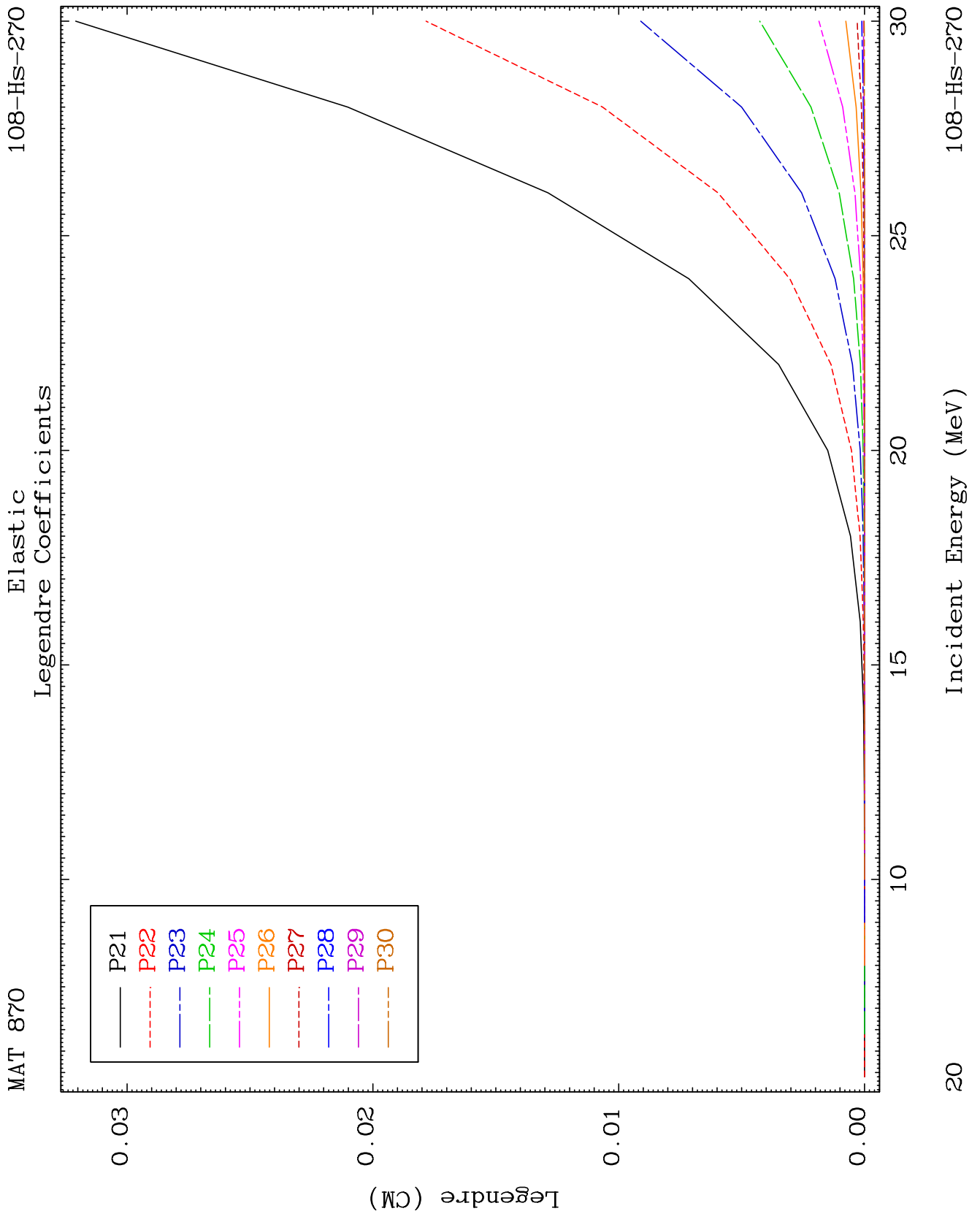
17

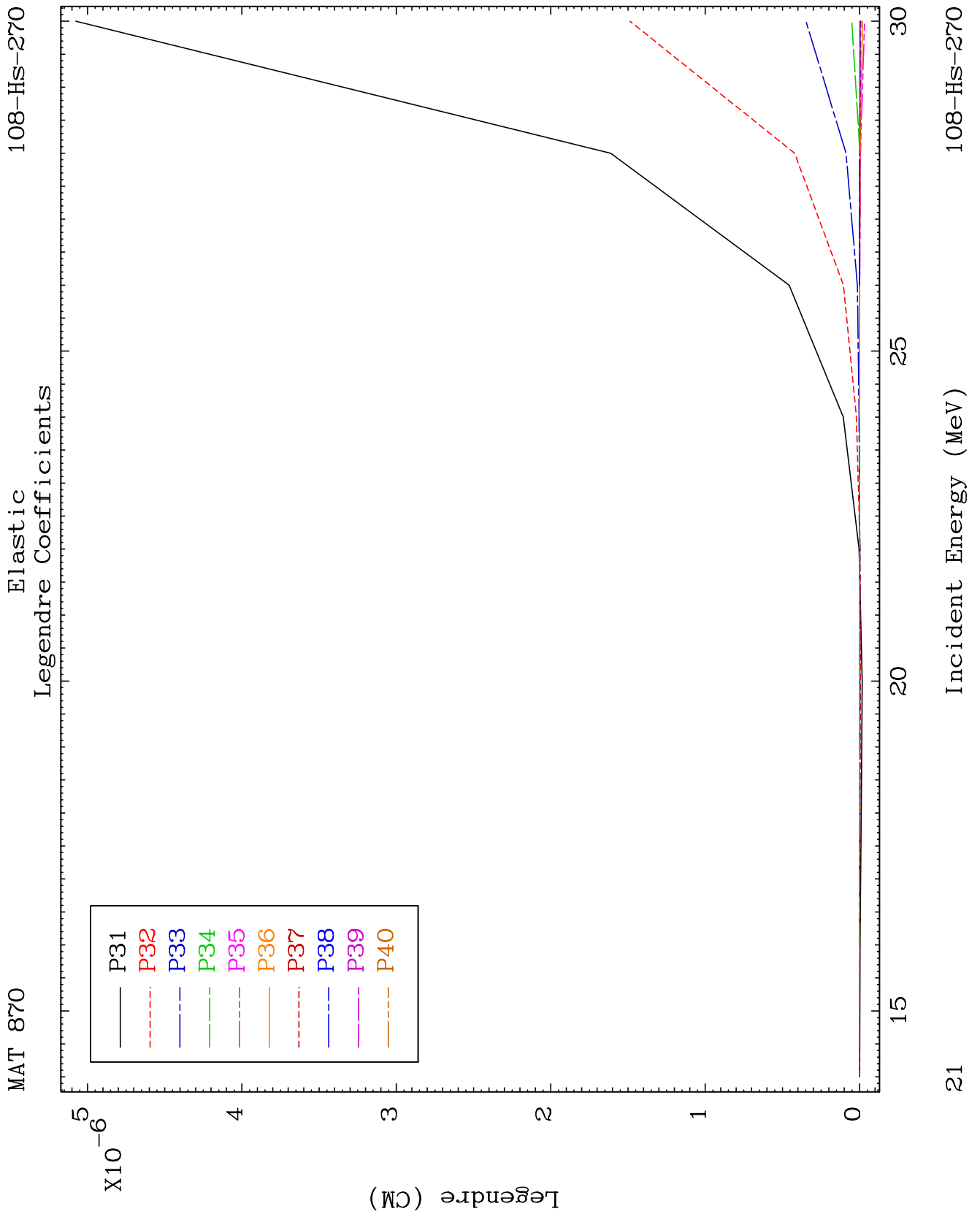
Incident Energy (MeV)

108-Hs-270





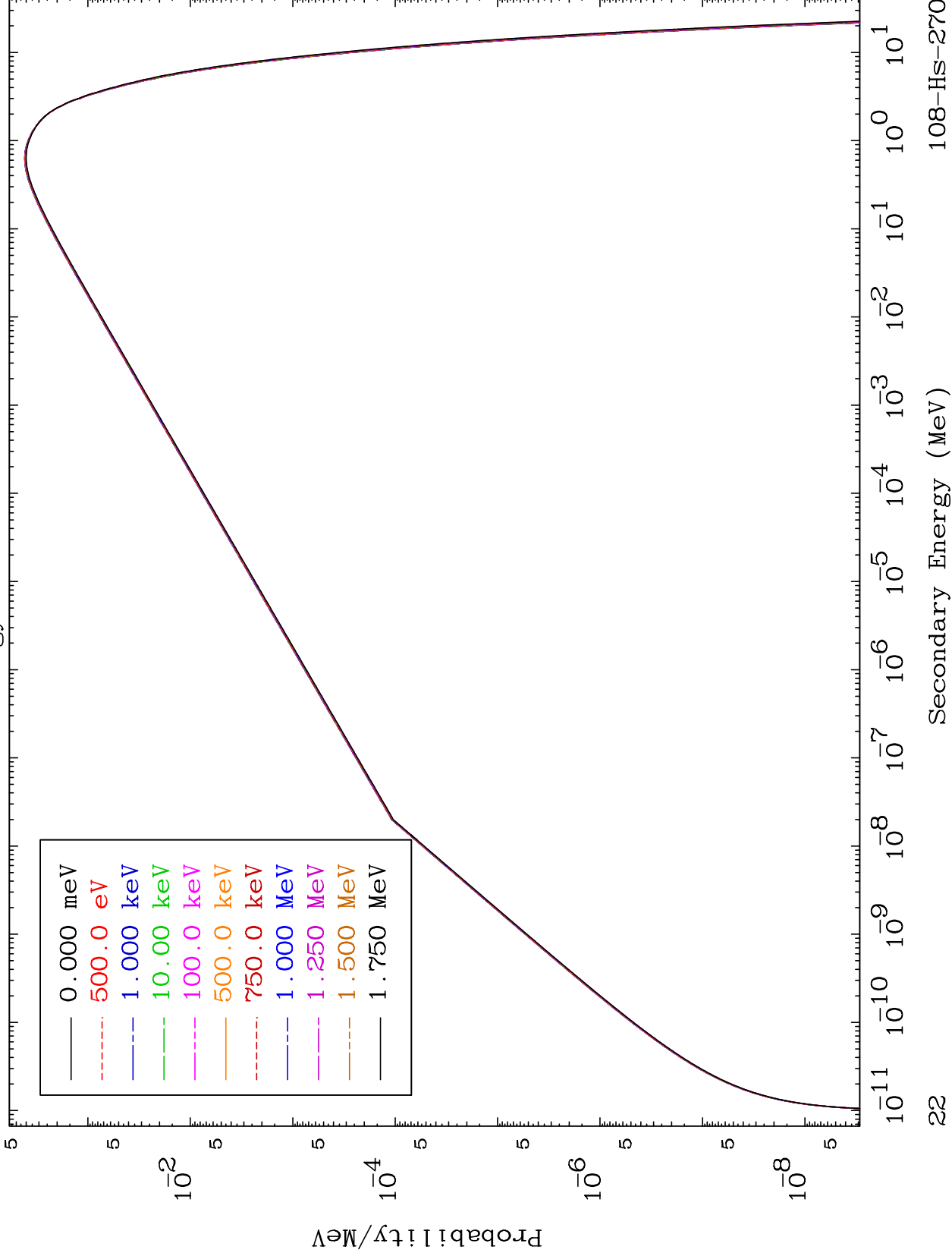




MAT 870

Fission Energy Distributions

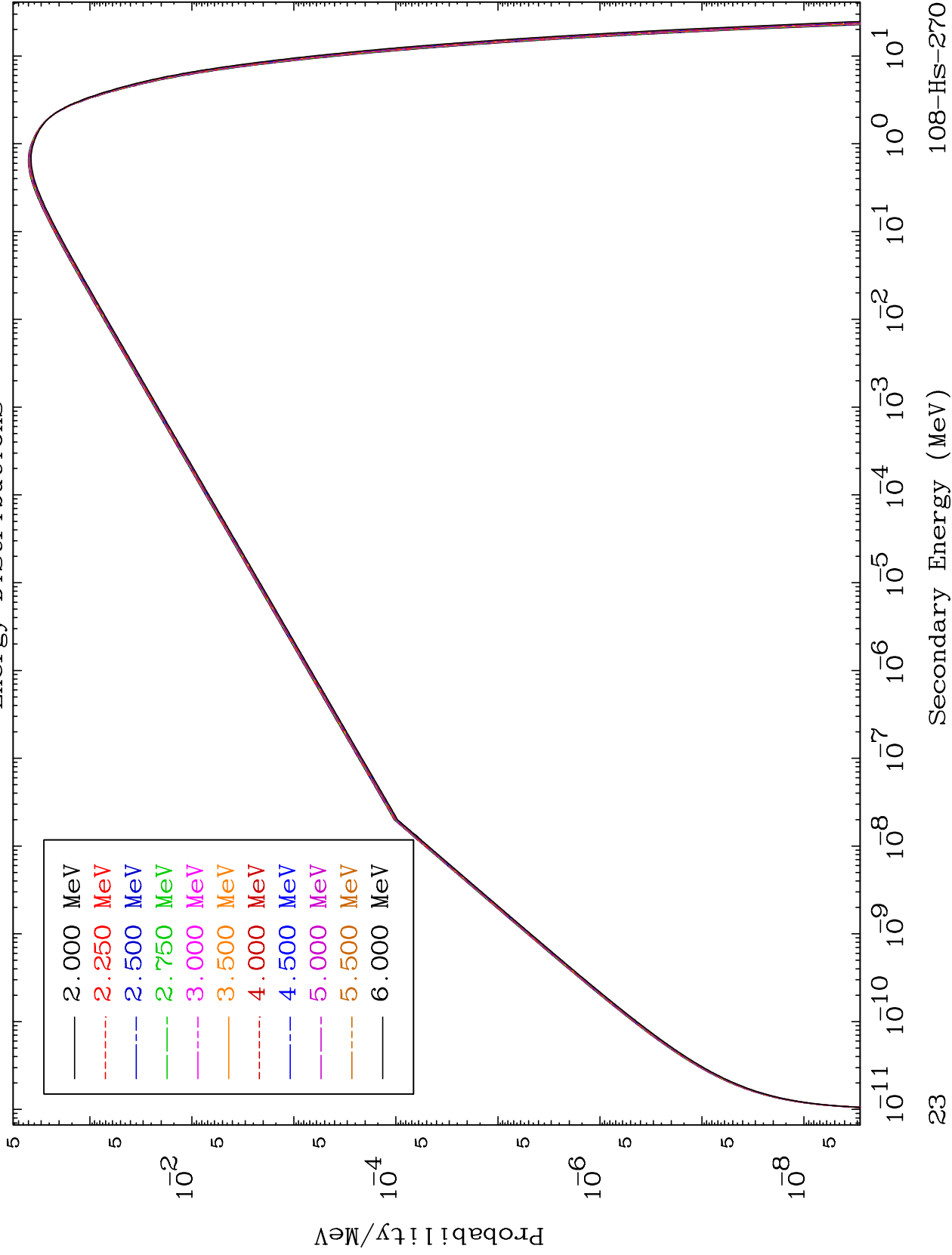
108-Hs-270



MAT 870

Fission Energy Distributions

108-Hs-270

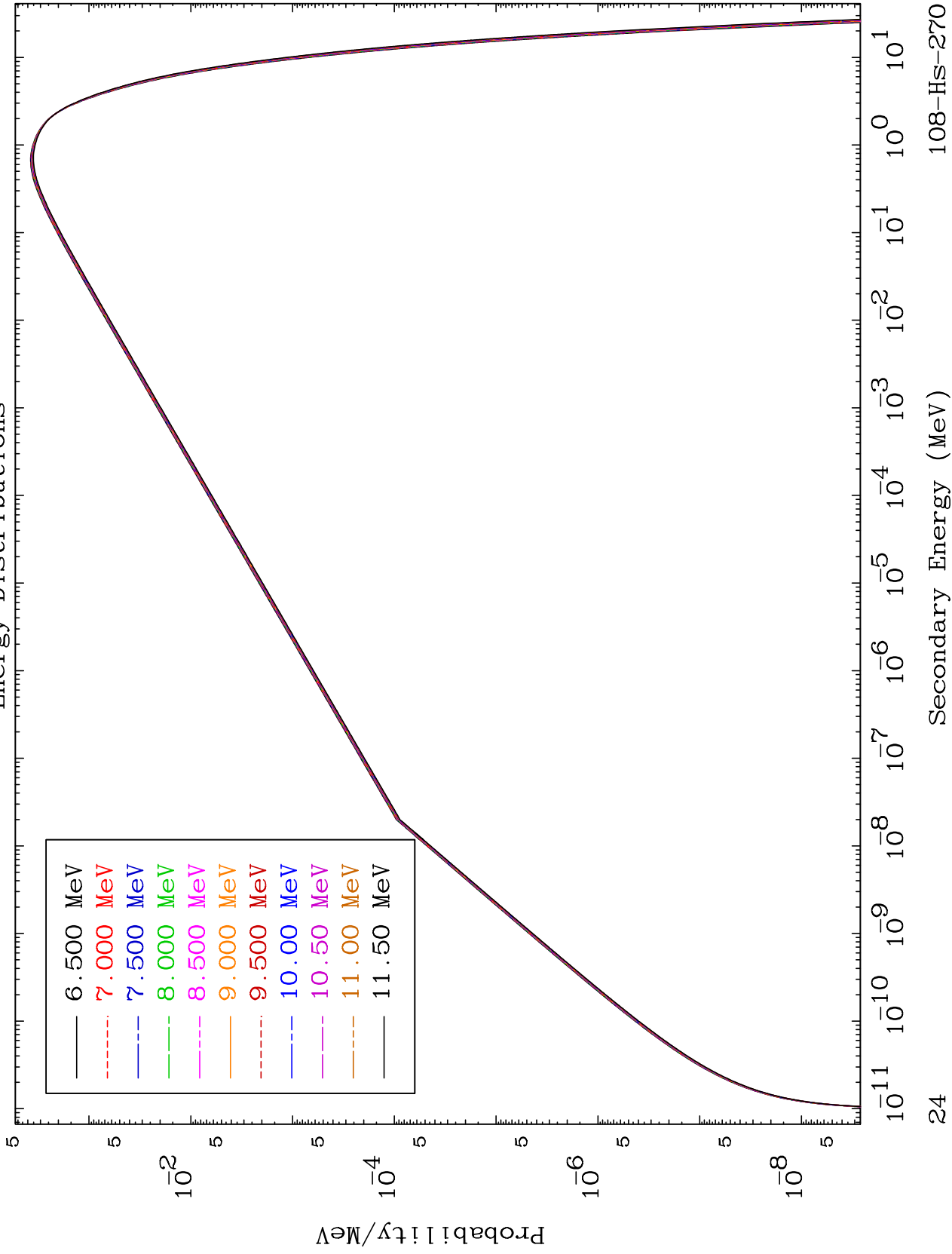


108-Hs-270

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Fission Energy Distributions

108-Hs-270

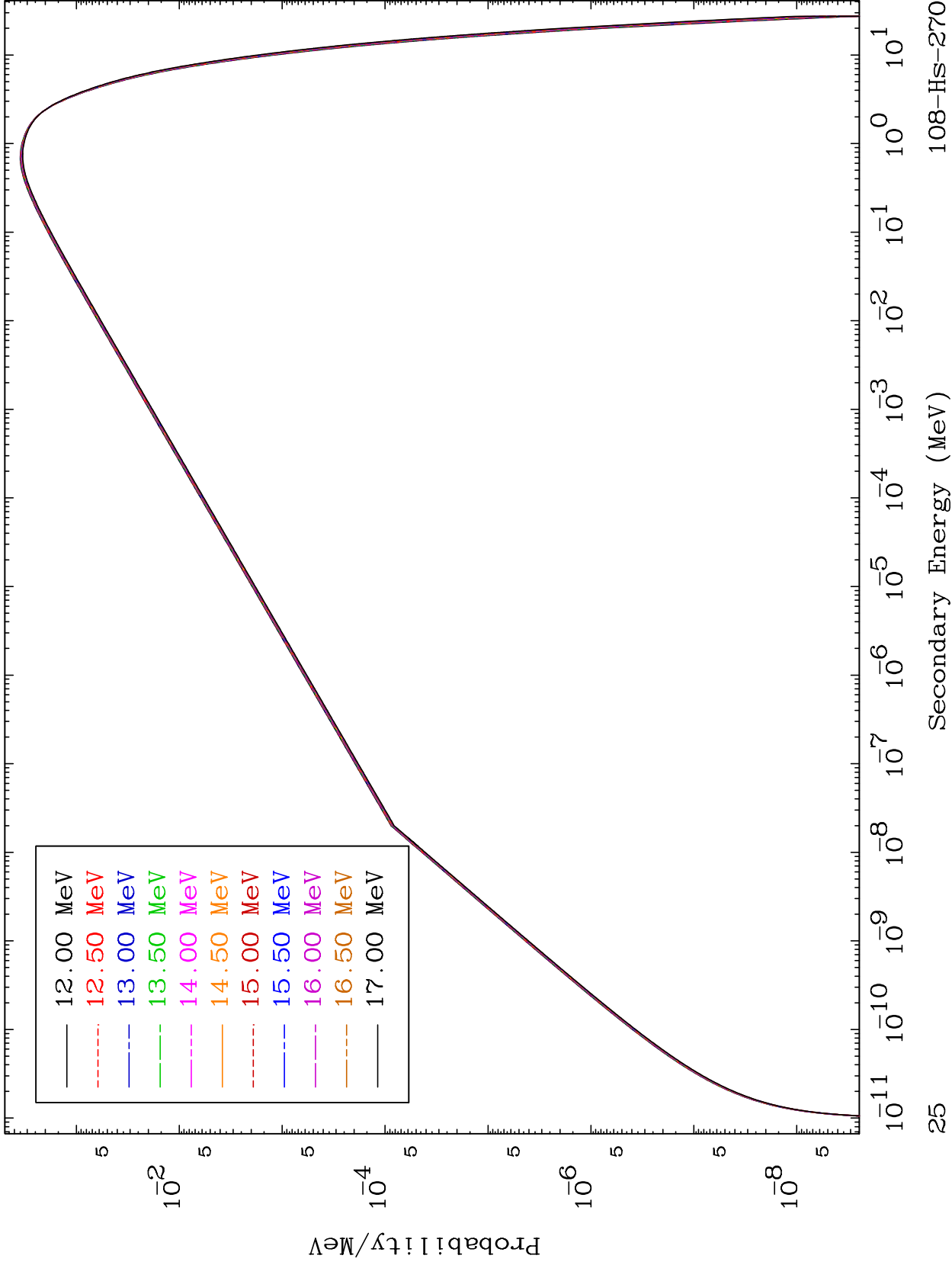


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Fission Energy Distributions

108-Hs-270



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Fission Energy Distributions

108-Hs-270

