

Program EVALPLOT
(Version 2021-1)

by

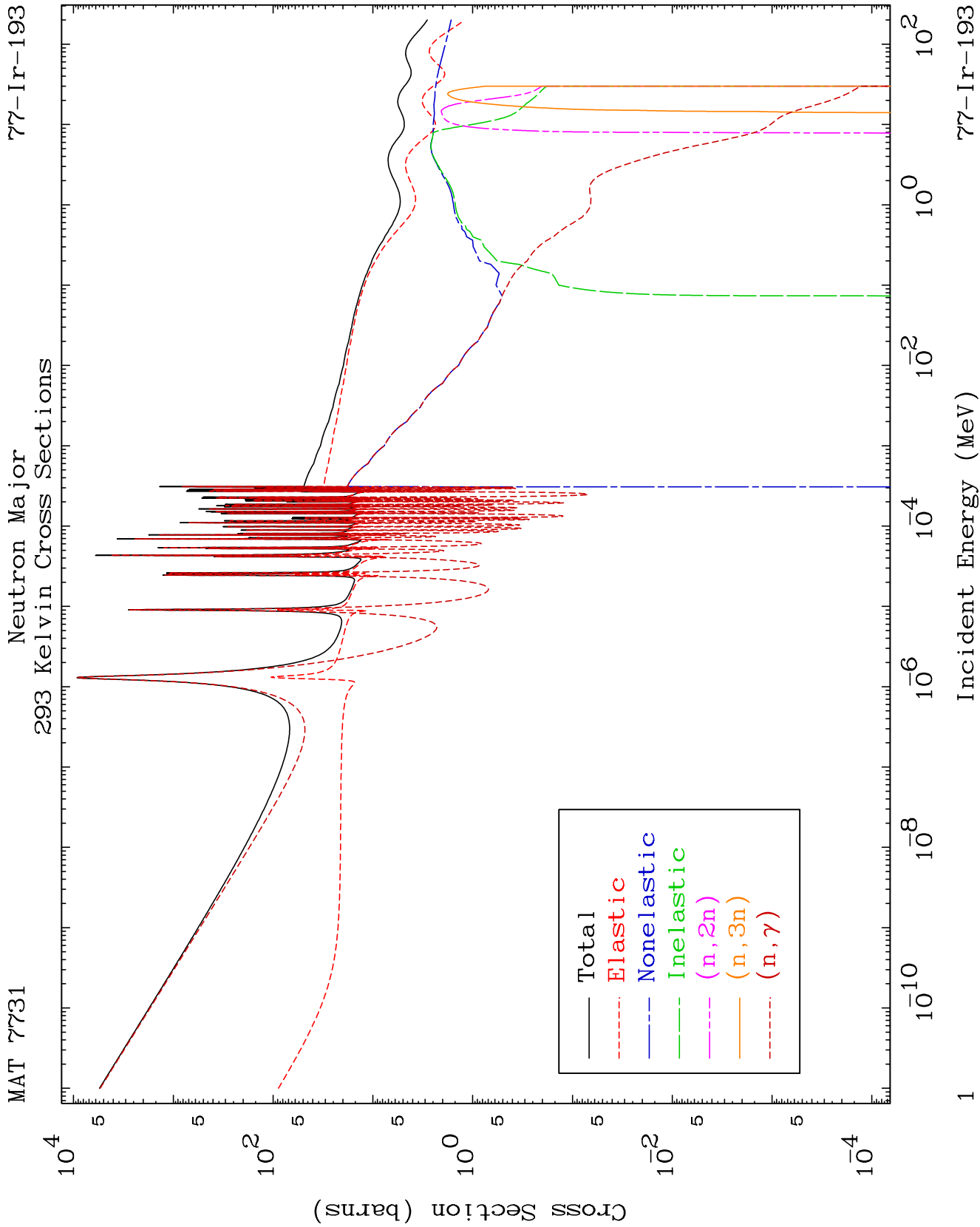
Dermott E. Cullen
(Present Contact Information)

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Tele: 925-443-1911

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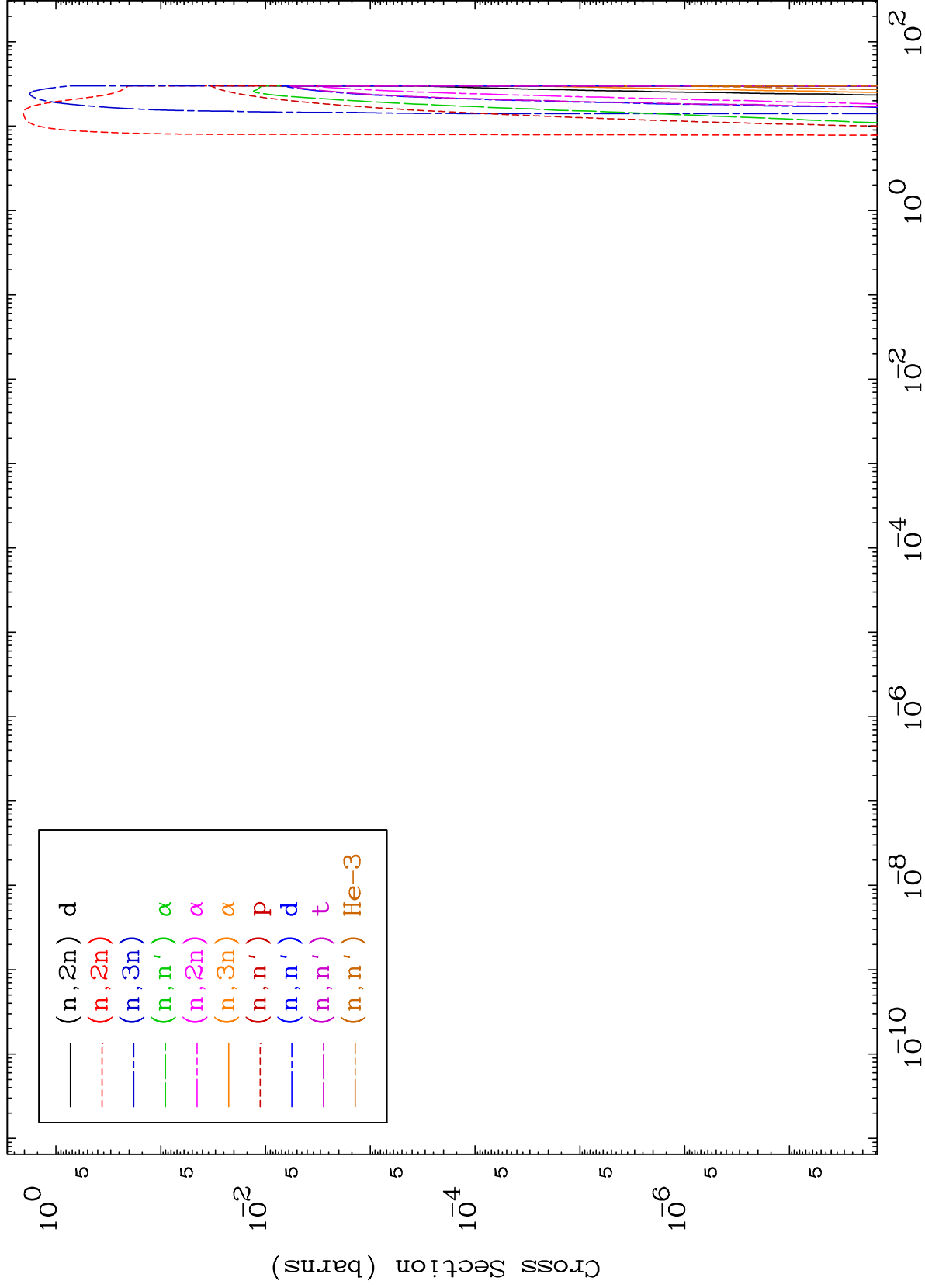
Press Mouse Button to Start

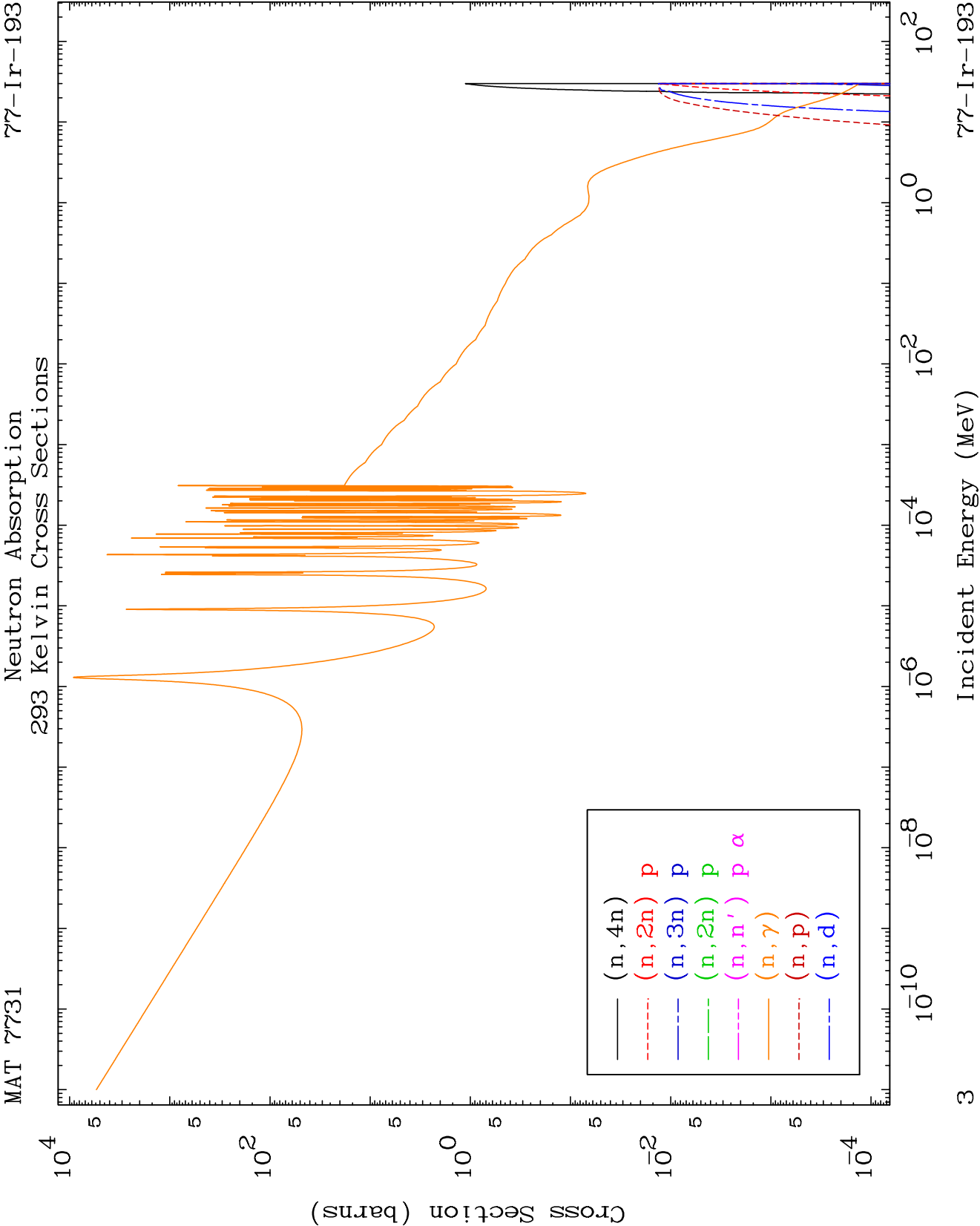


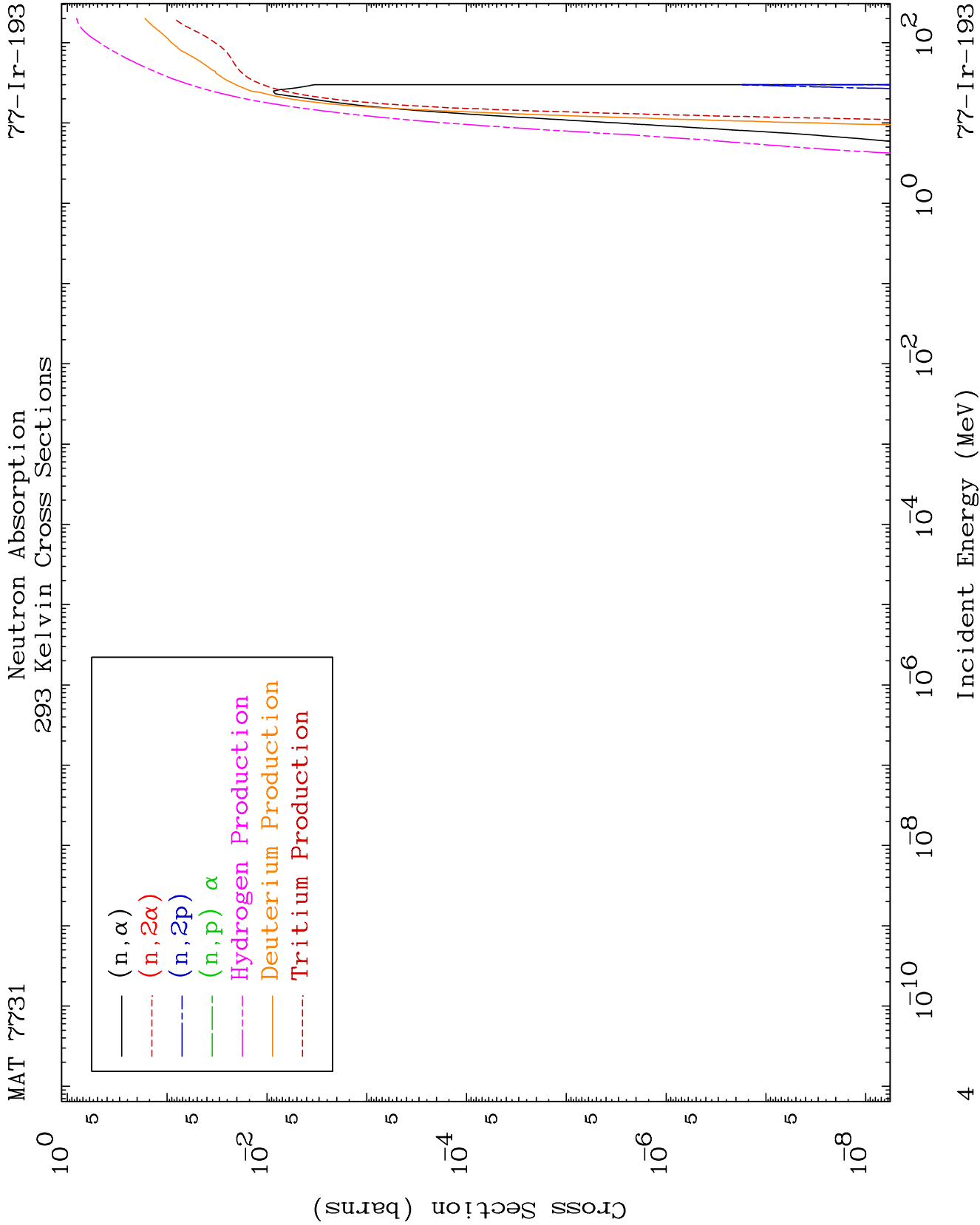
MAT 7731

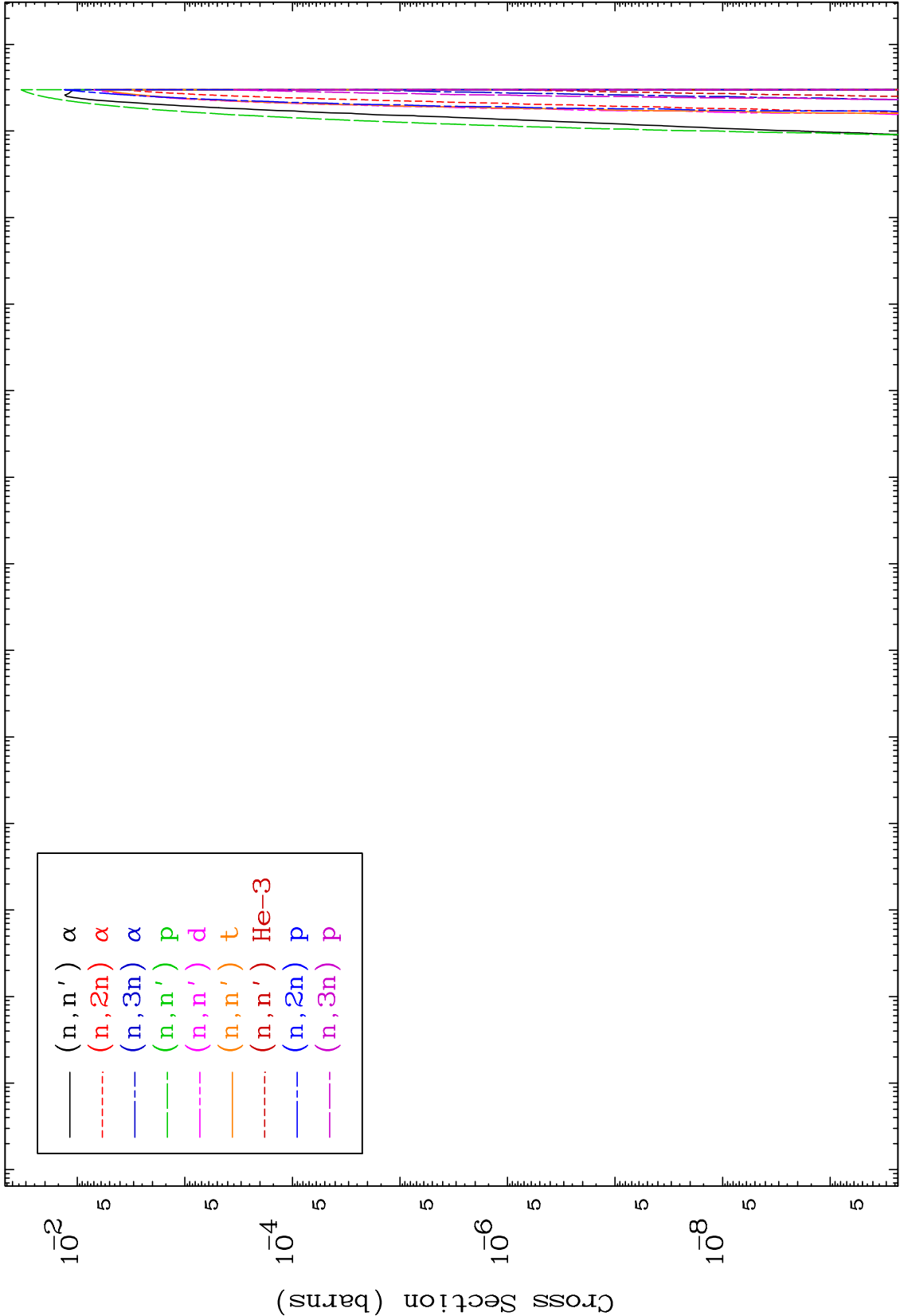
Neutron Absorption
293 Kelvin Cross Sections

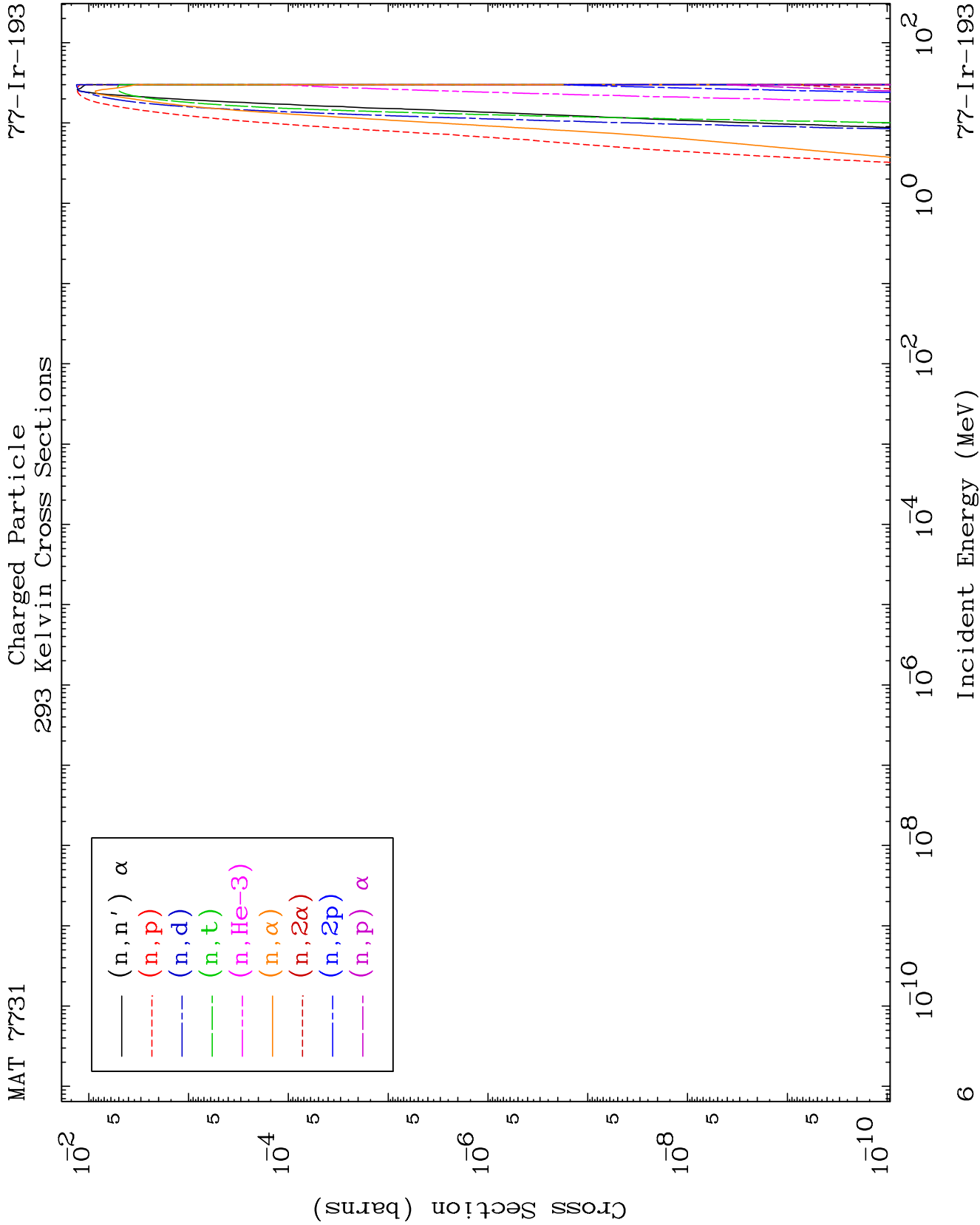
77-Ir-193

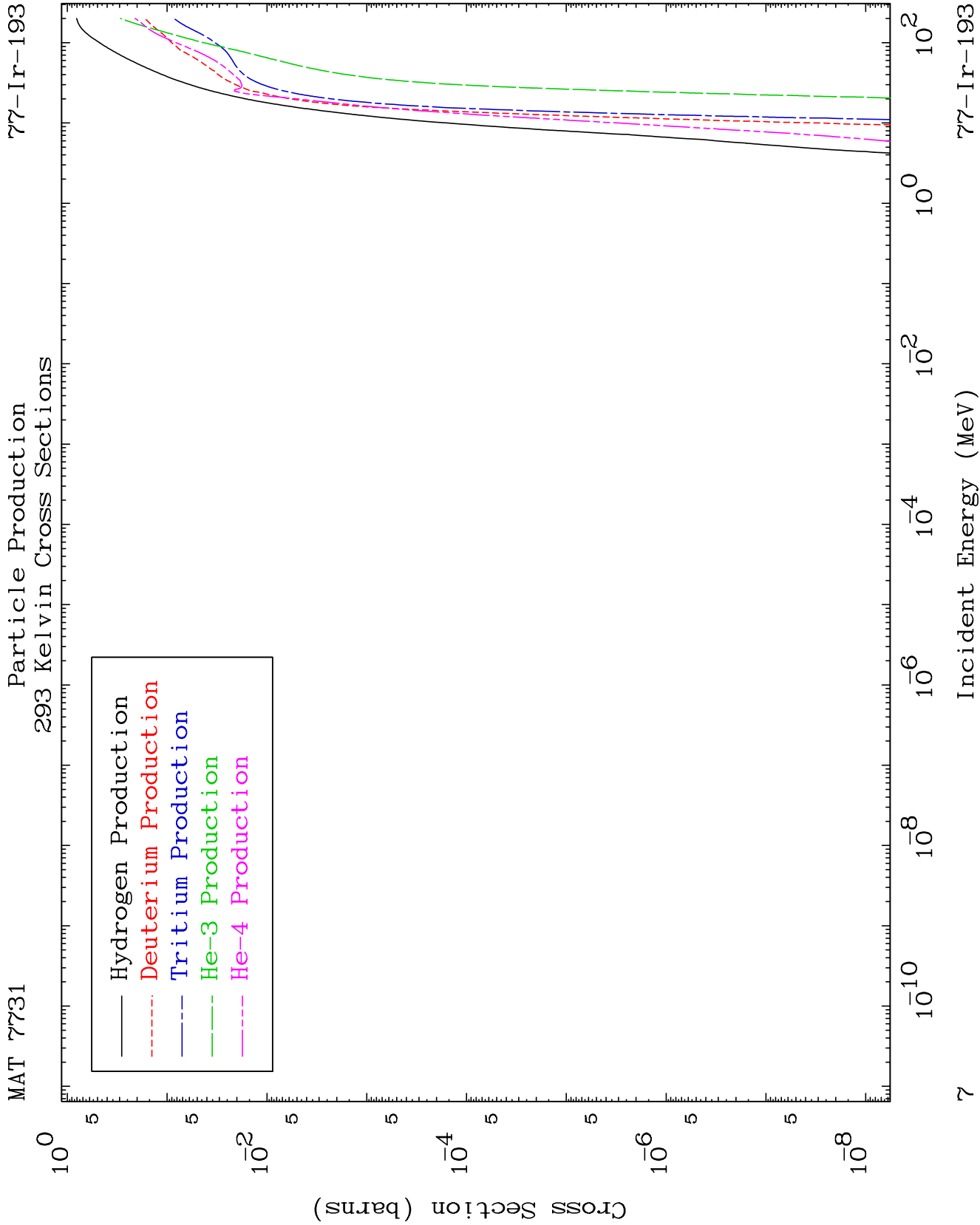


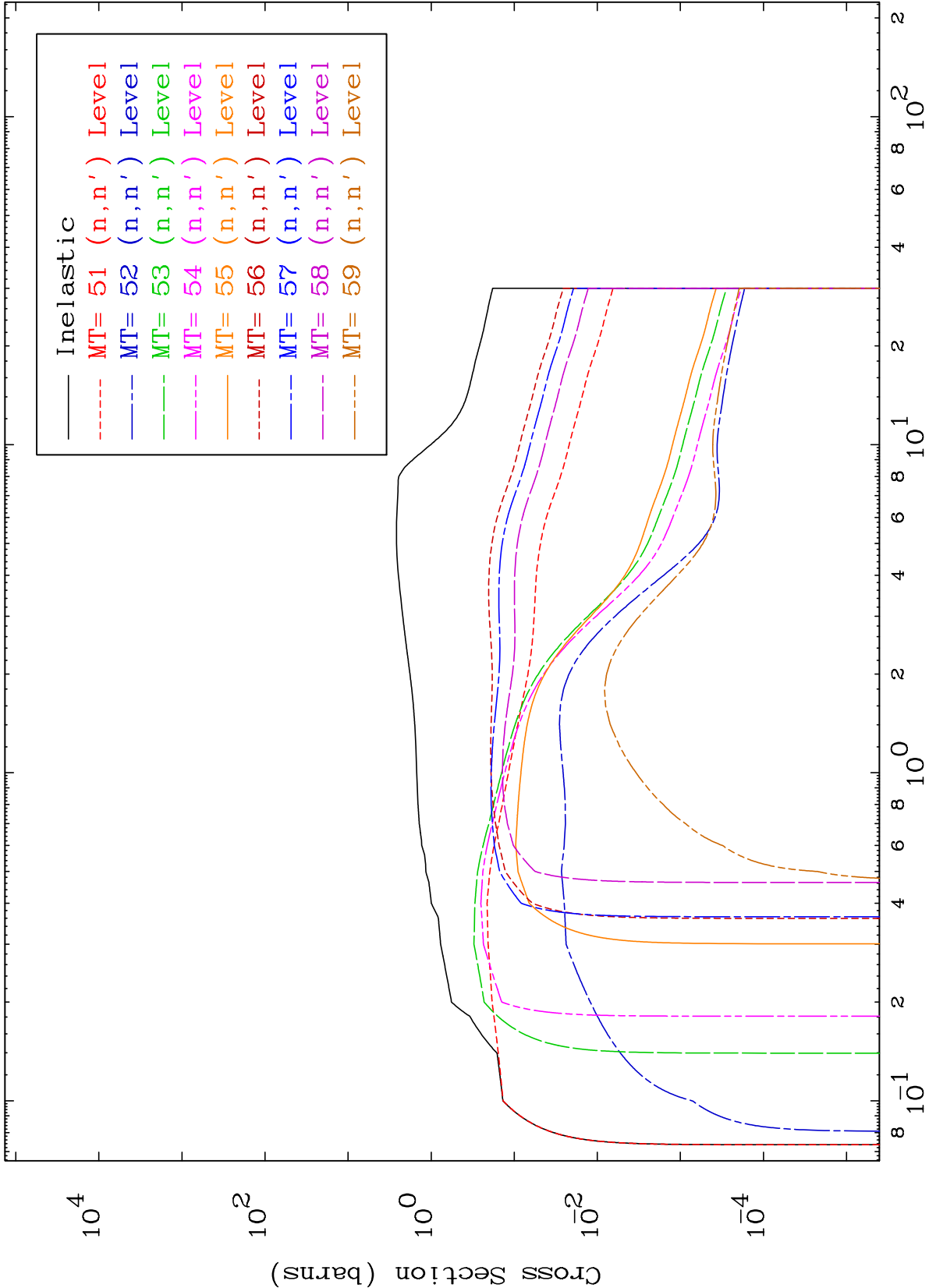


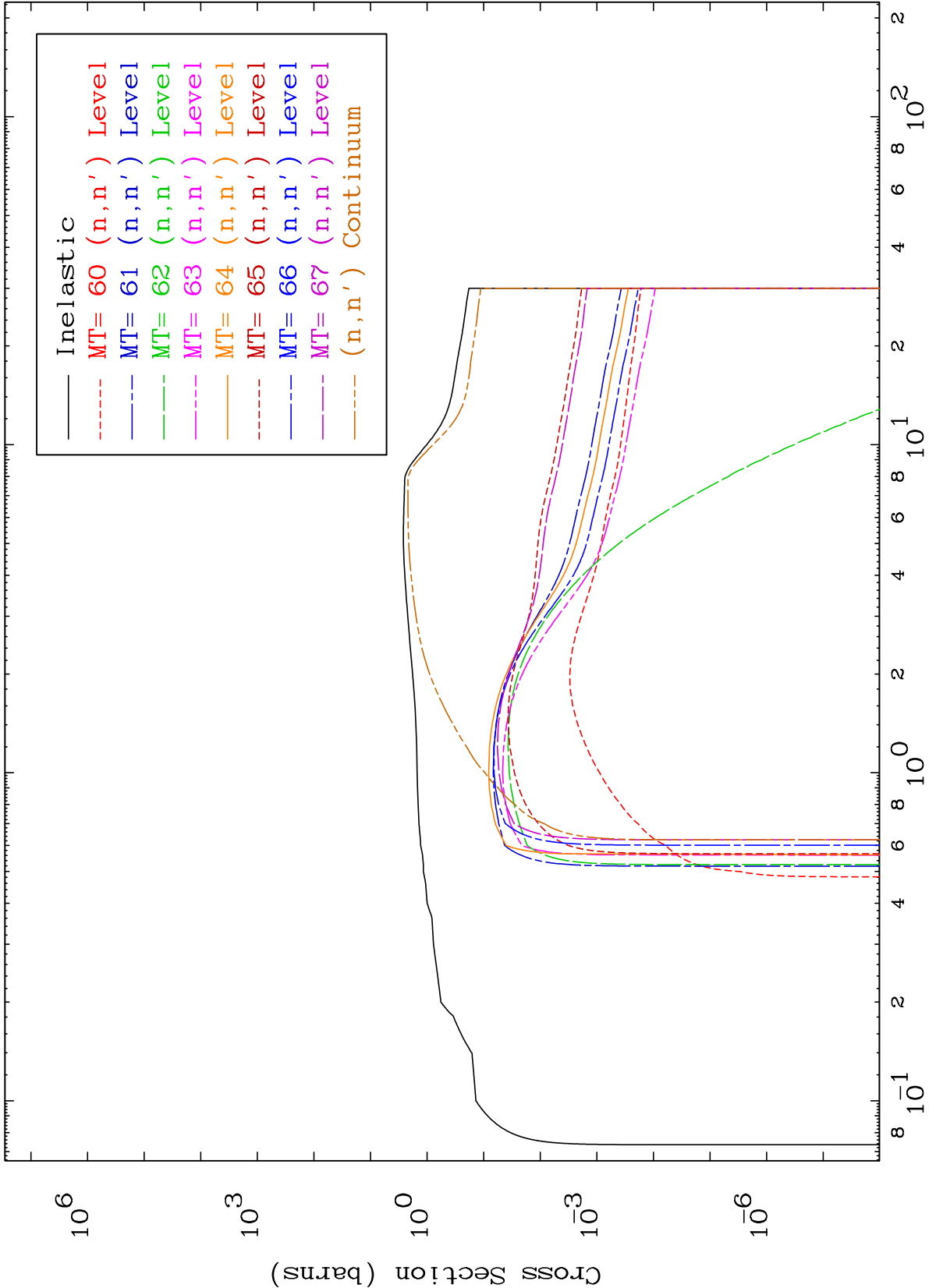


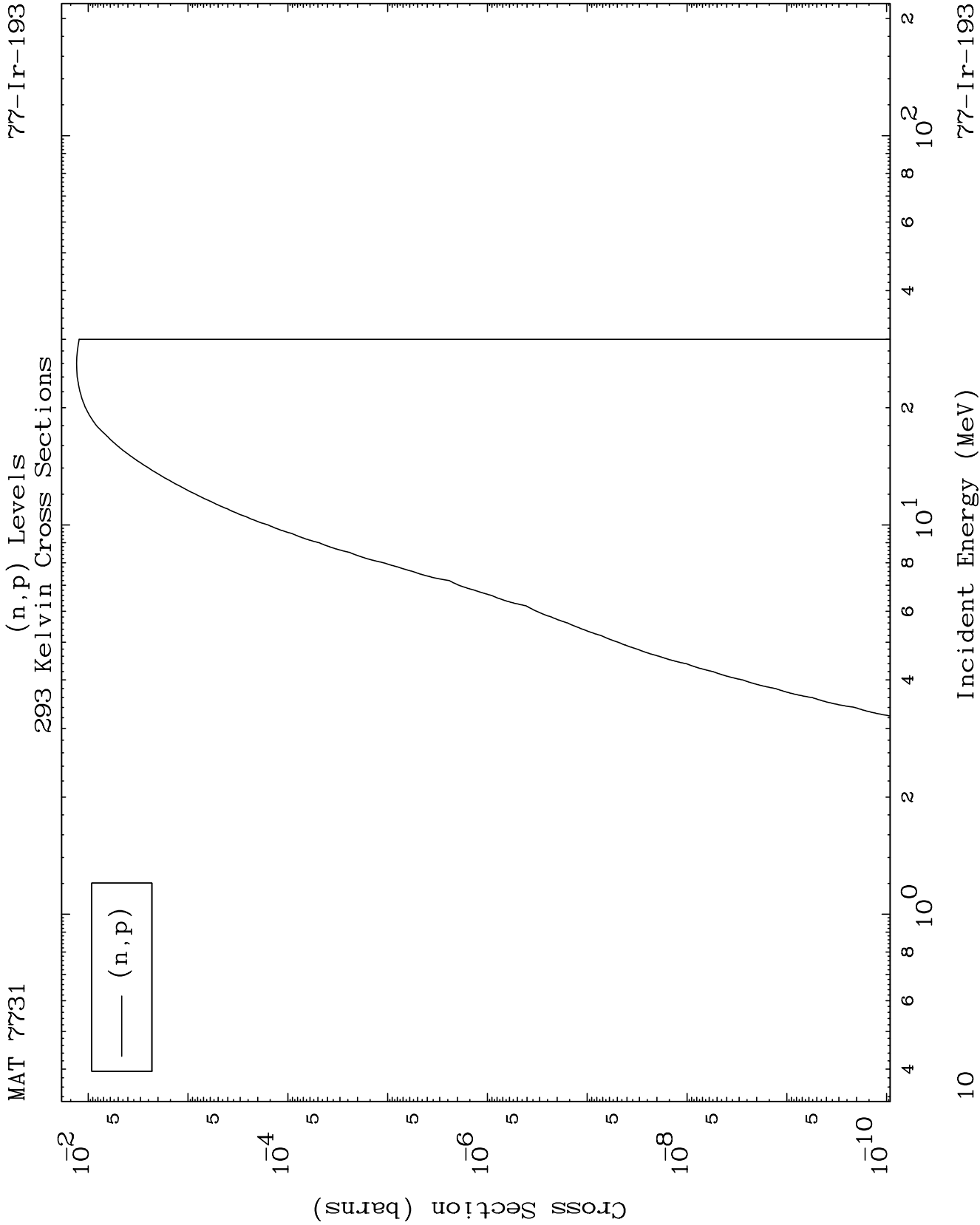








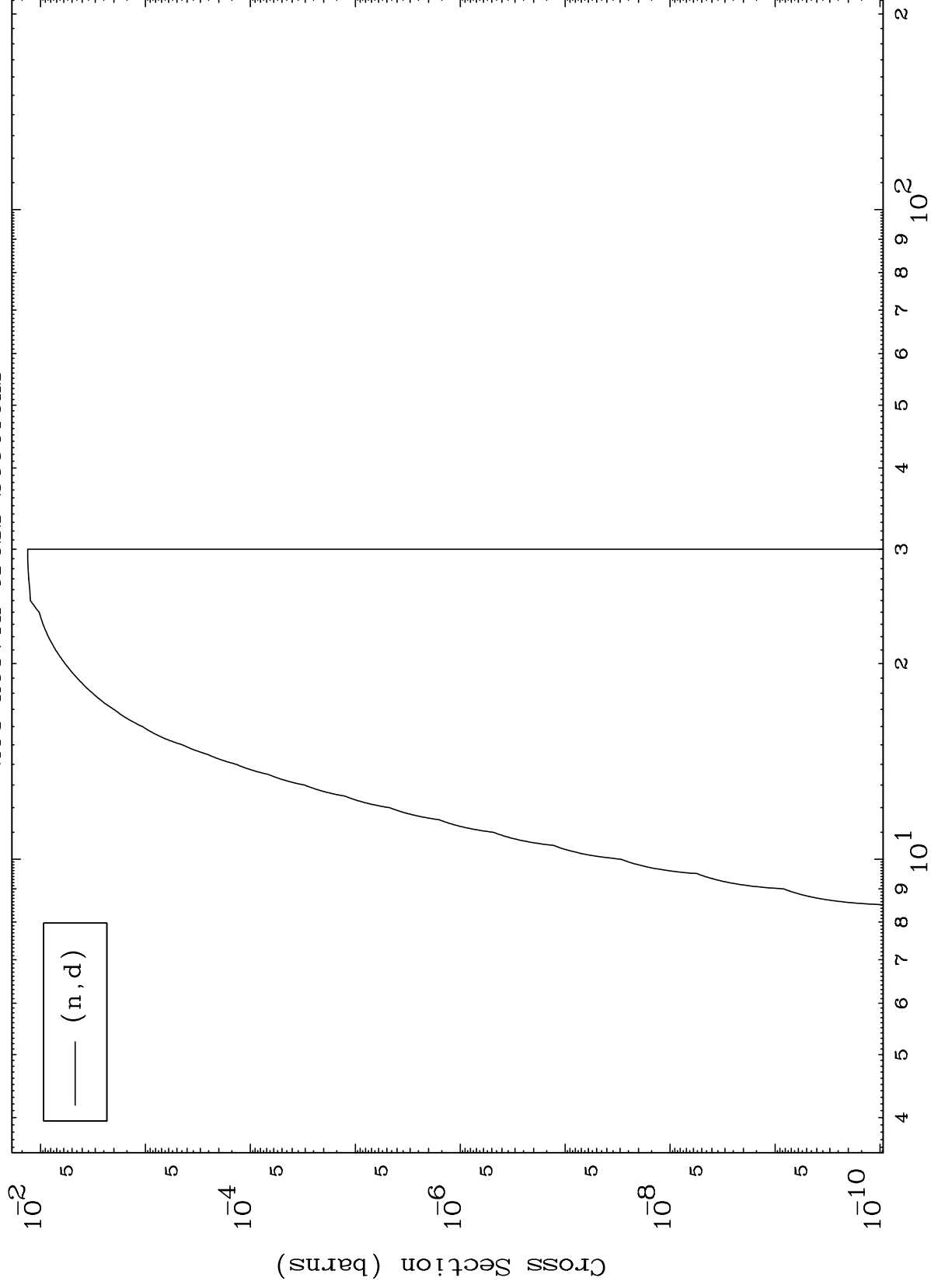




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77-Ir-193

(n,d) Levels
293 Kelvin Cross Sections



11

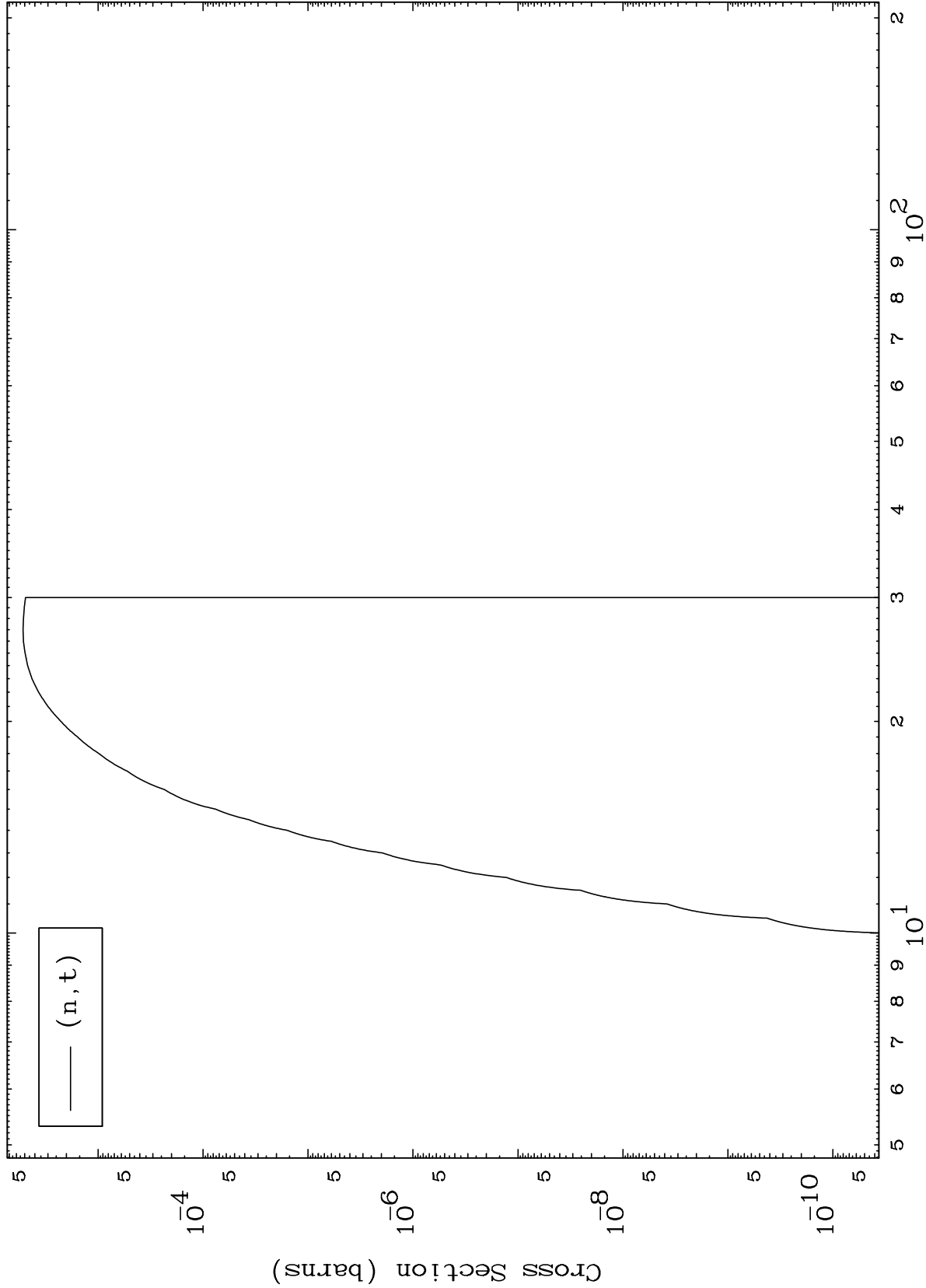
Incident Energy (MeV)

77-Ir-193

MAT 7731

77-Ir-193

(n,t) Levels
293 Kelvin Cross Sections



77-Ir-193

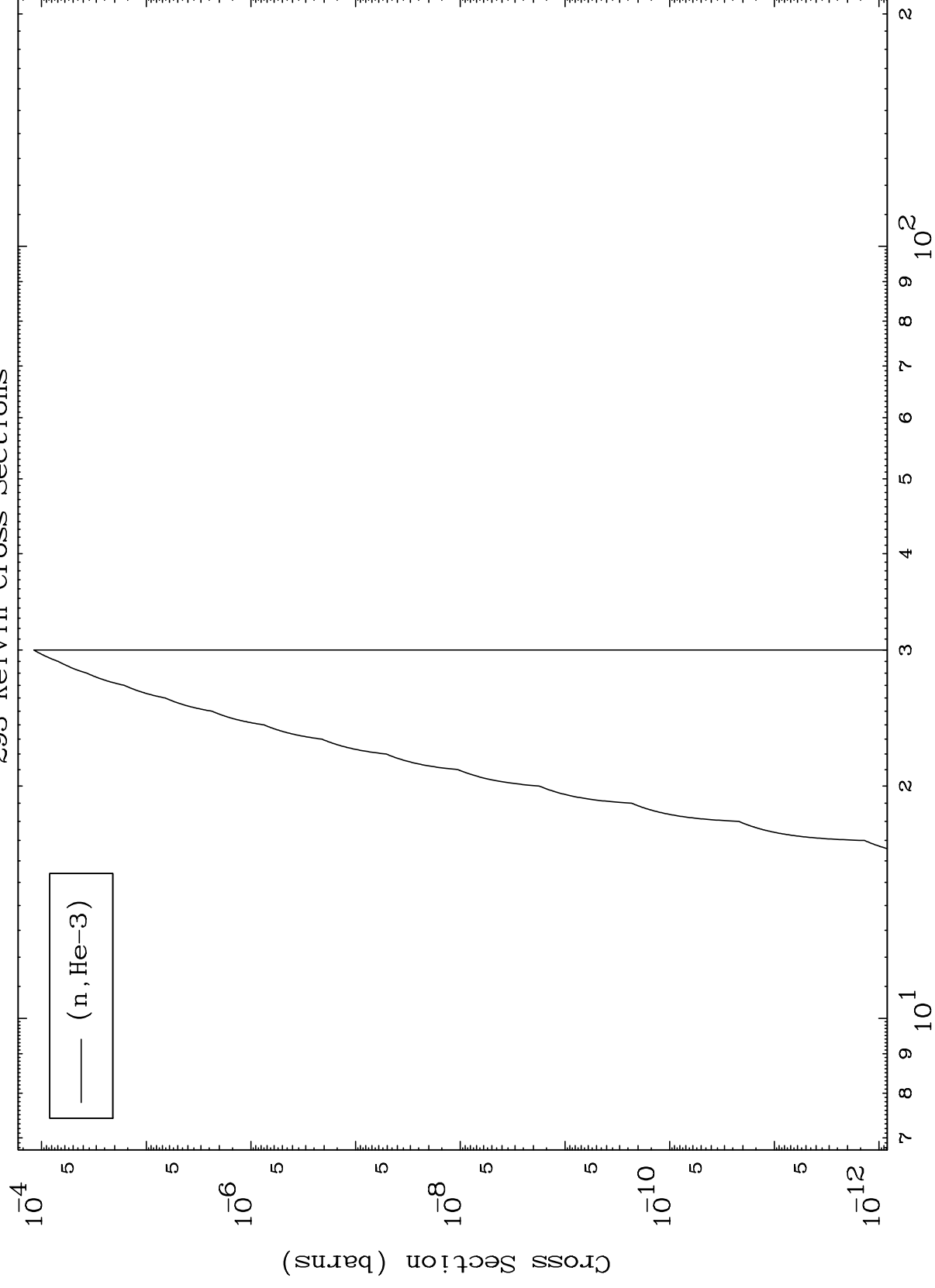
Incident Energy (MeV)

12

MAT 7731

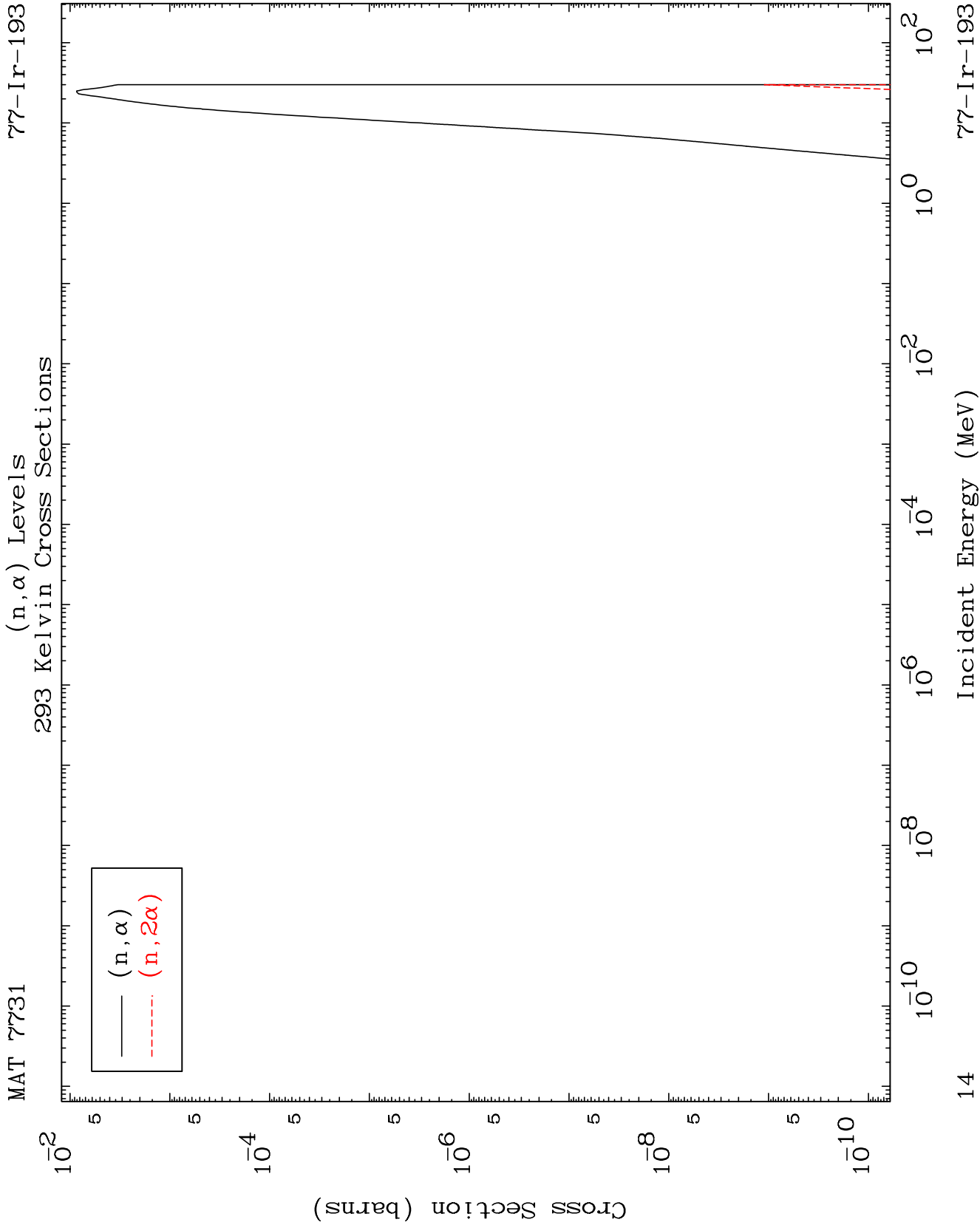
77-Ir-193

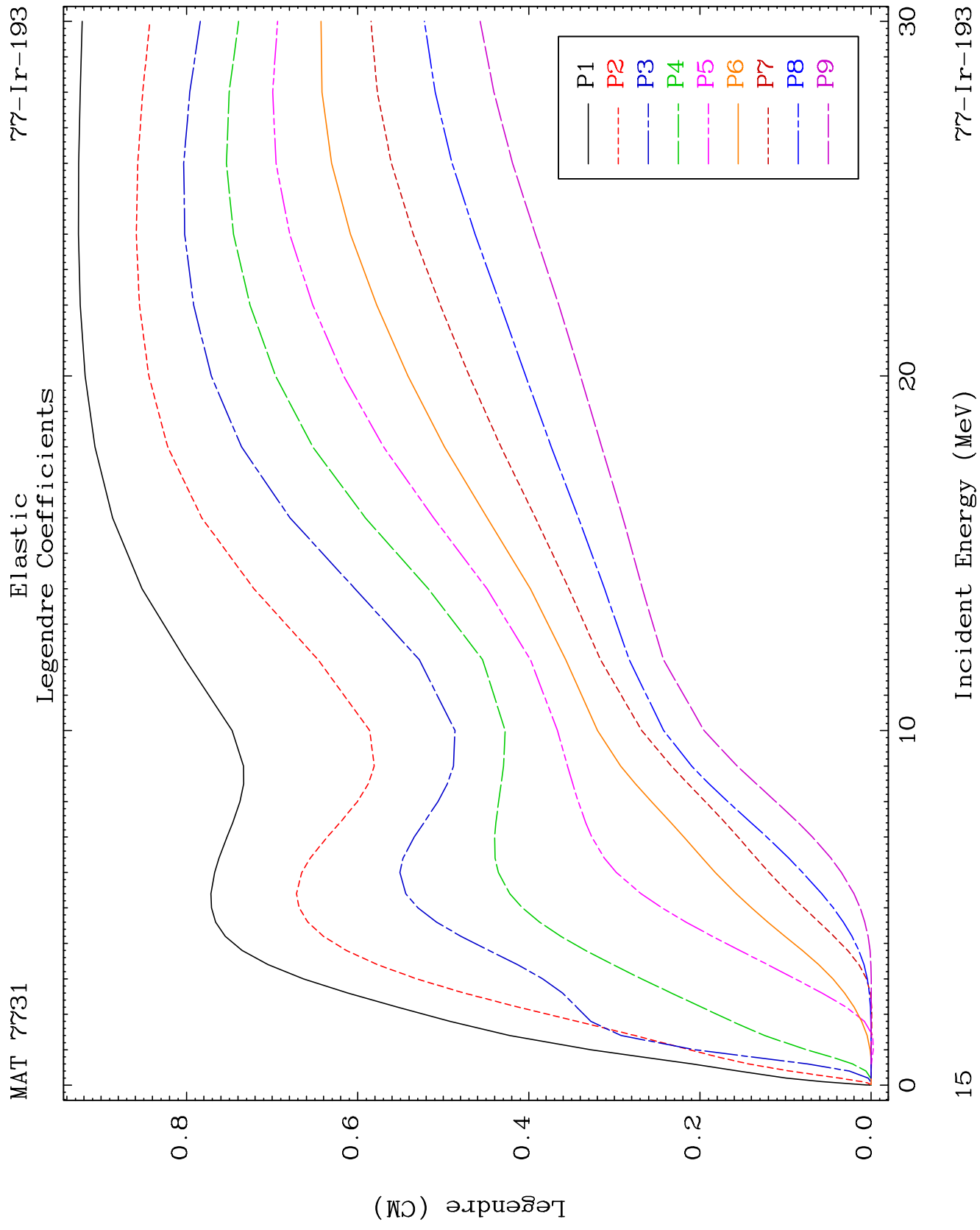
(n, He3) Levels
293 Kelvin Cross Sections

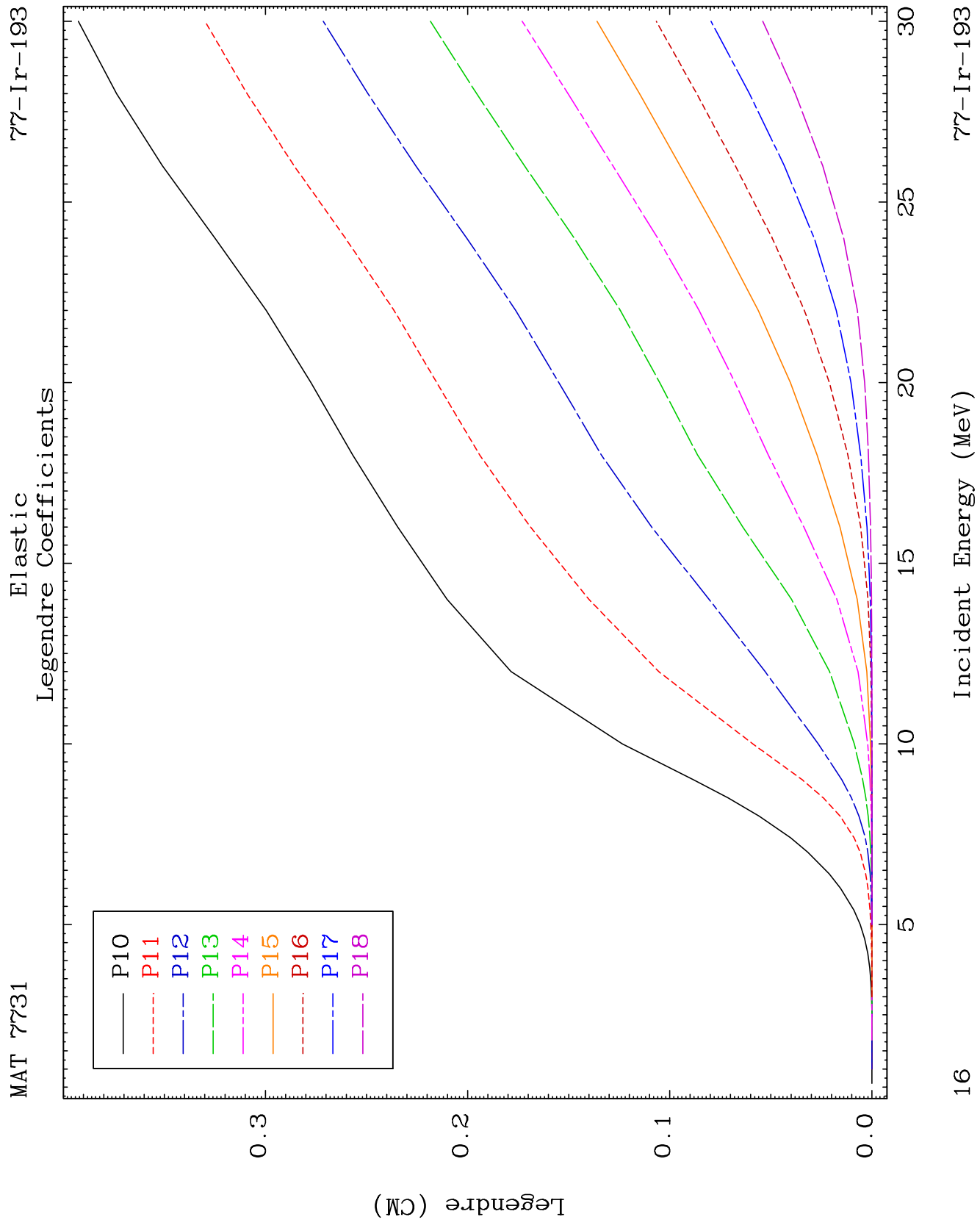


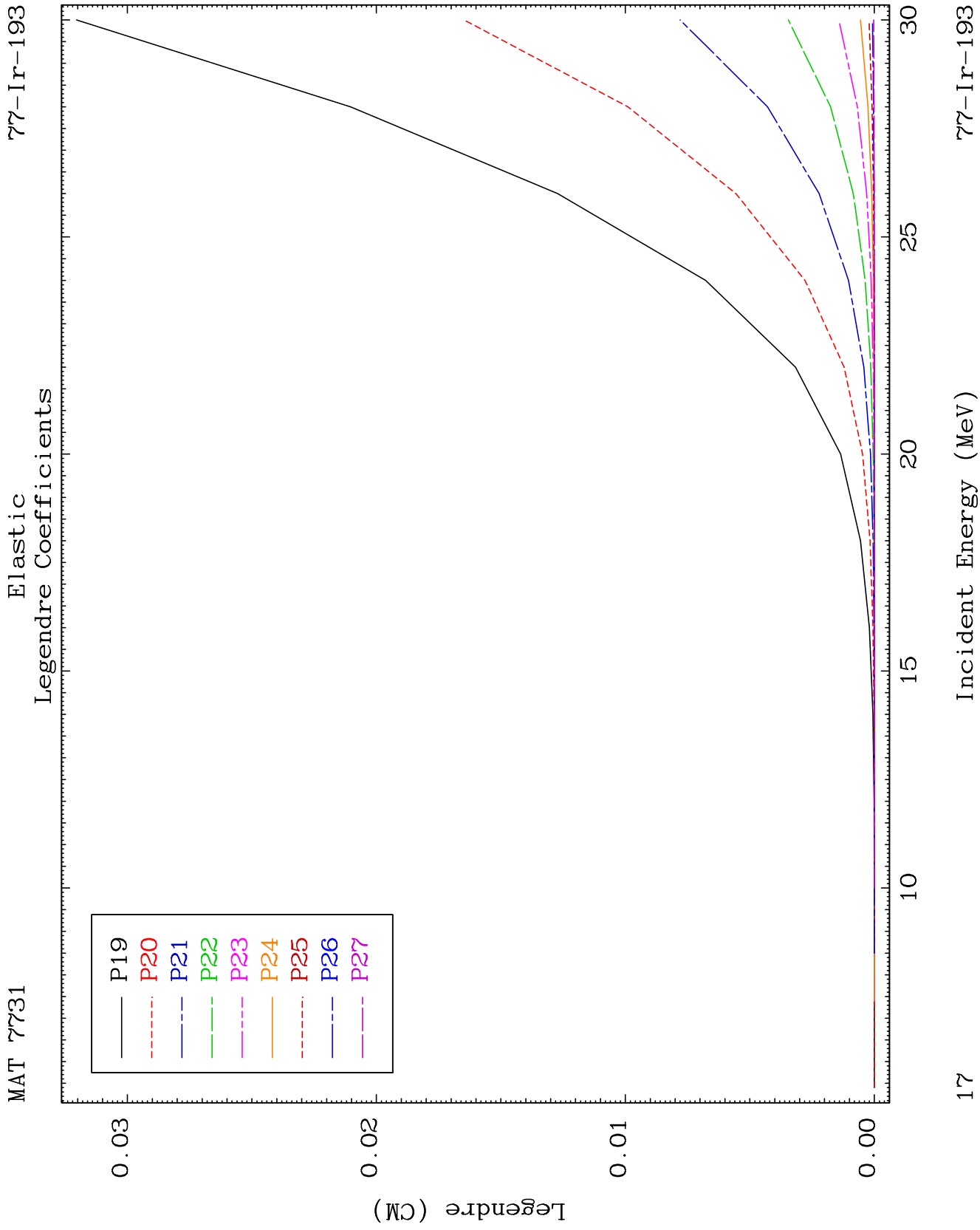
77-Ir-193

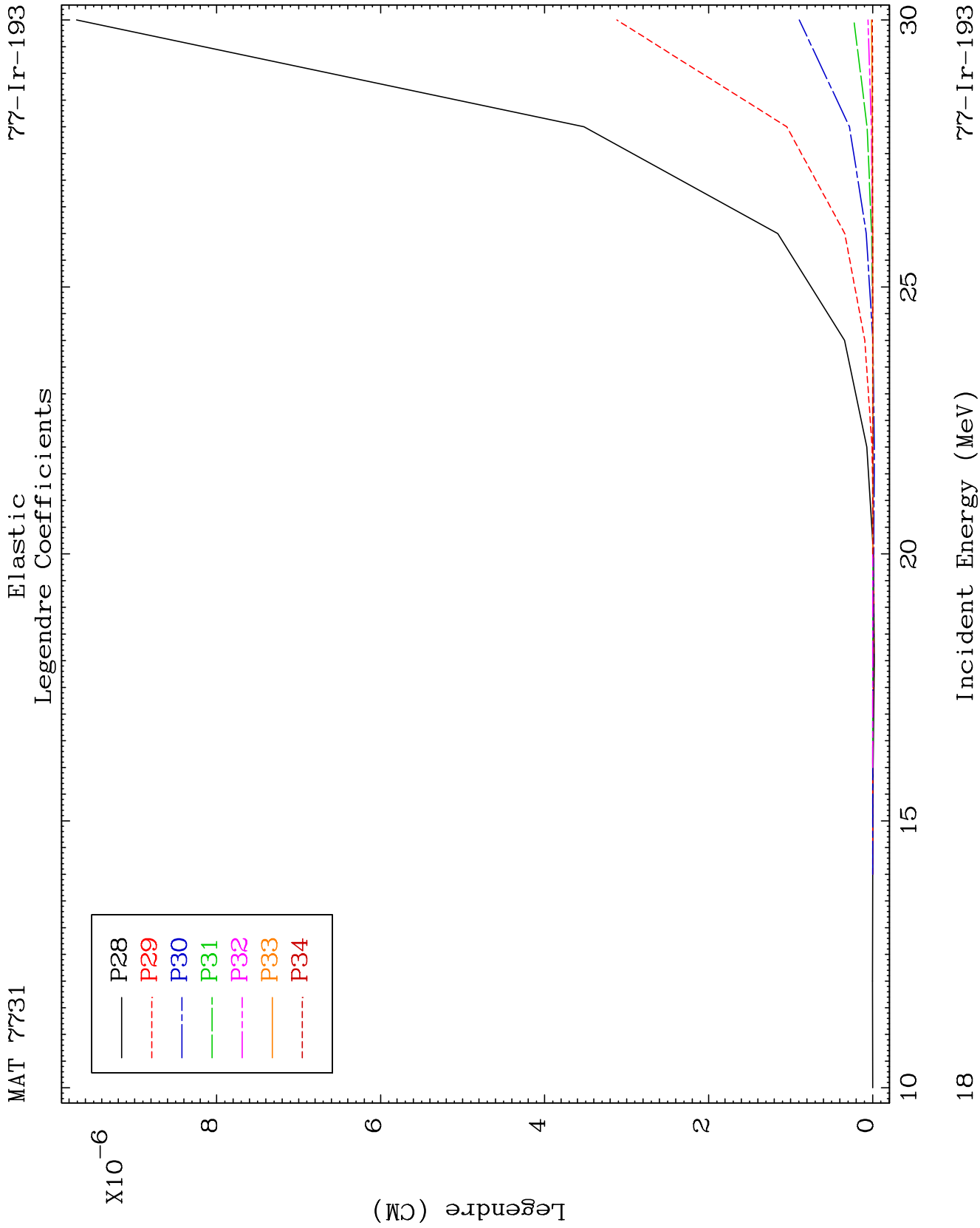
Incident Energy (MeV)

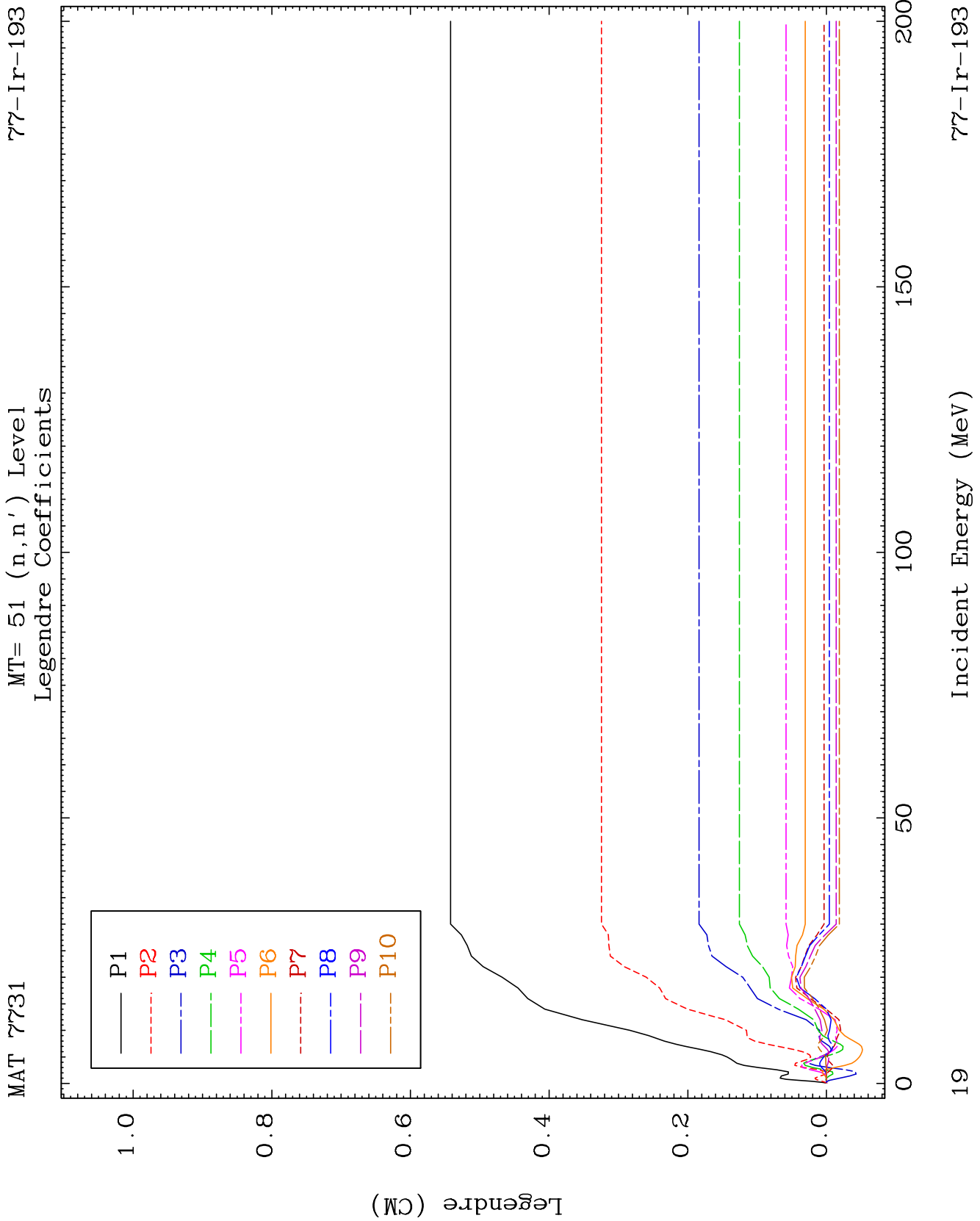








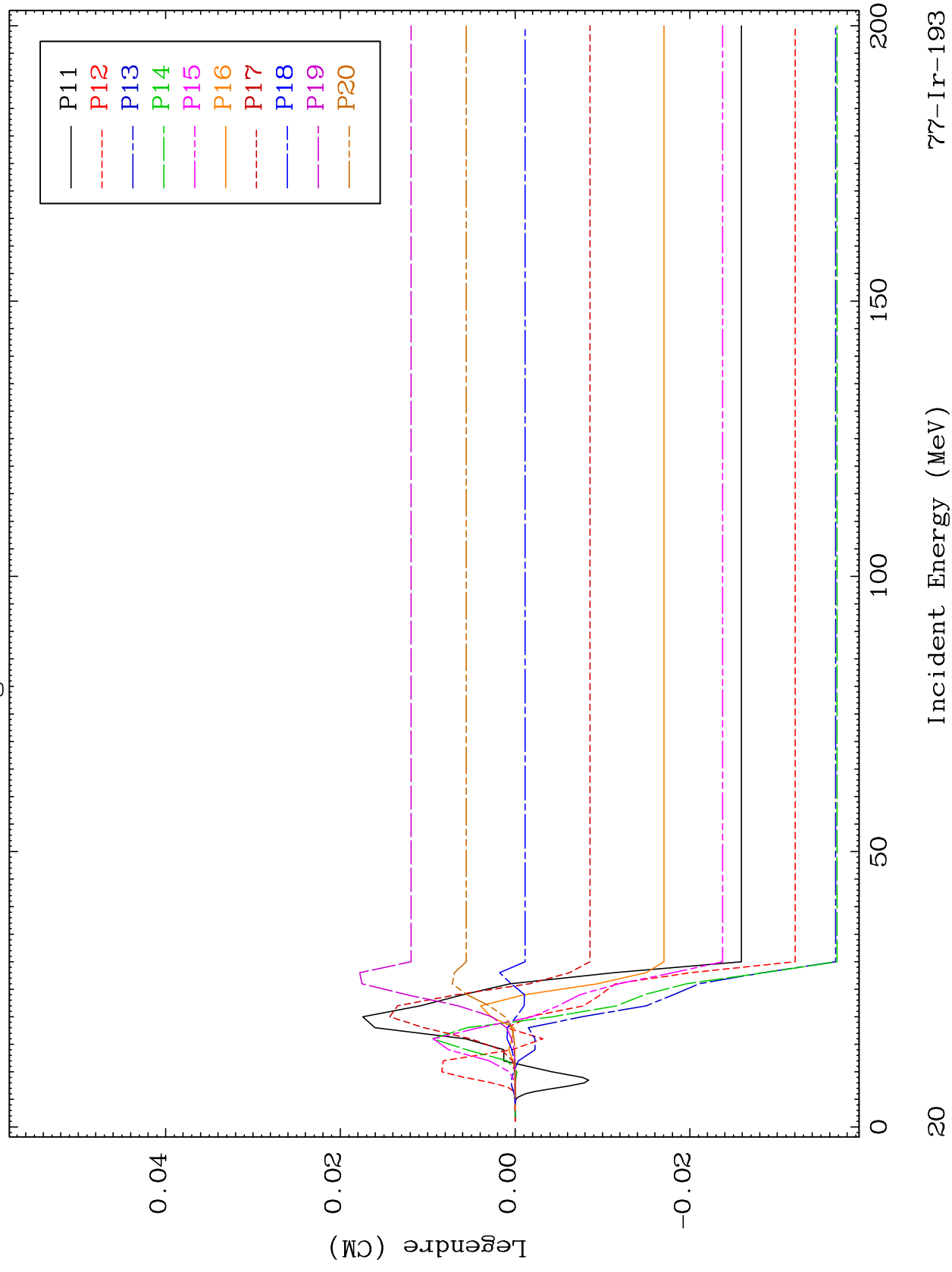


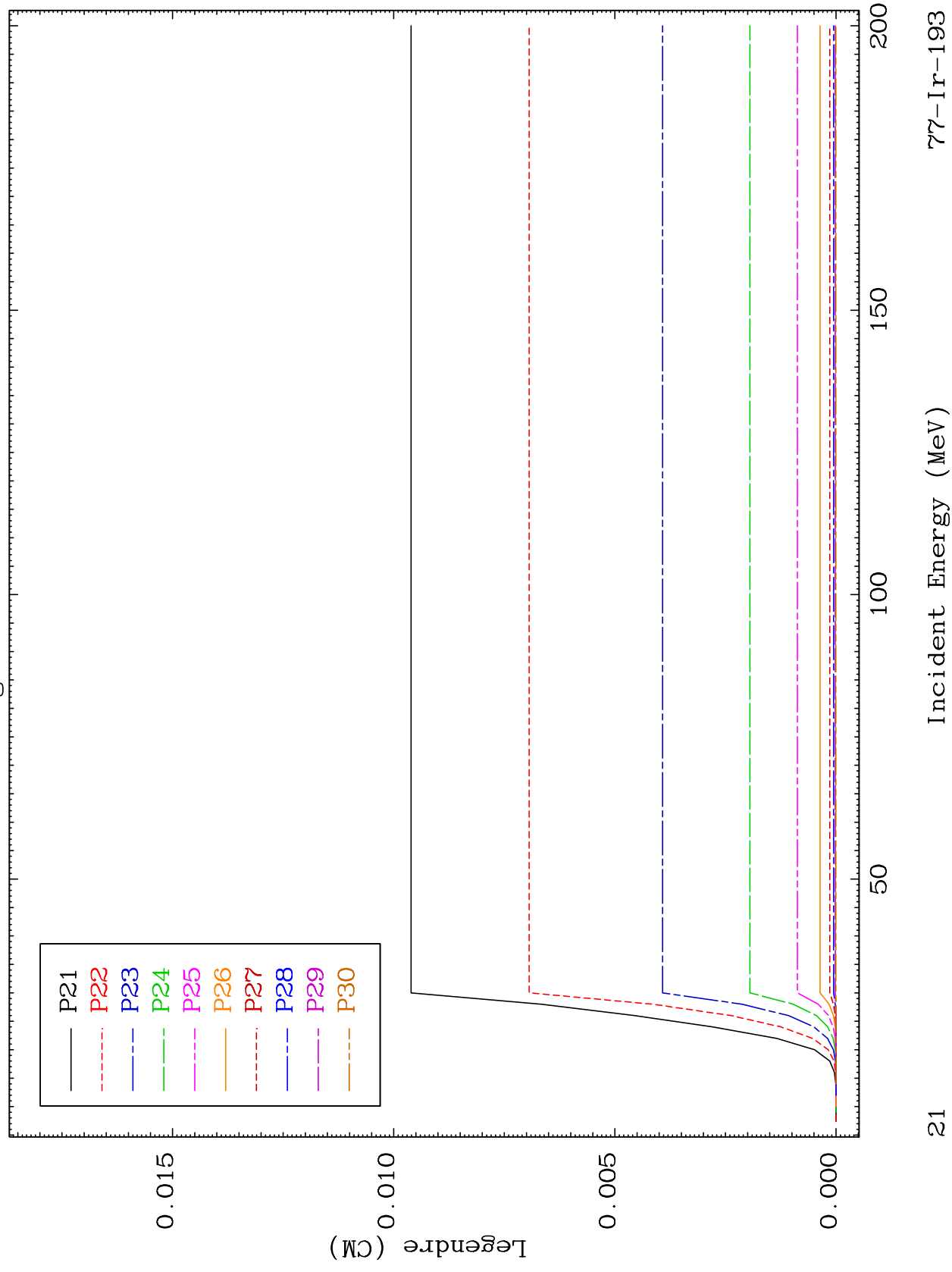


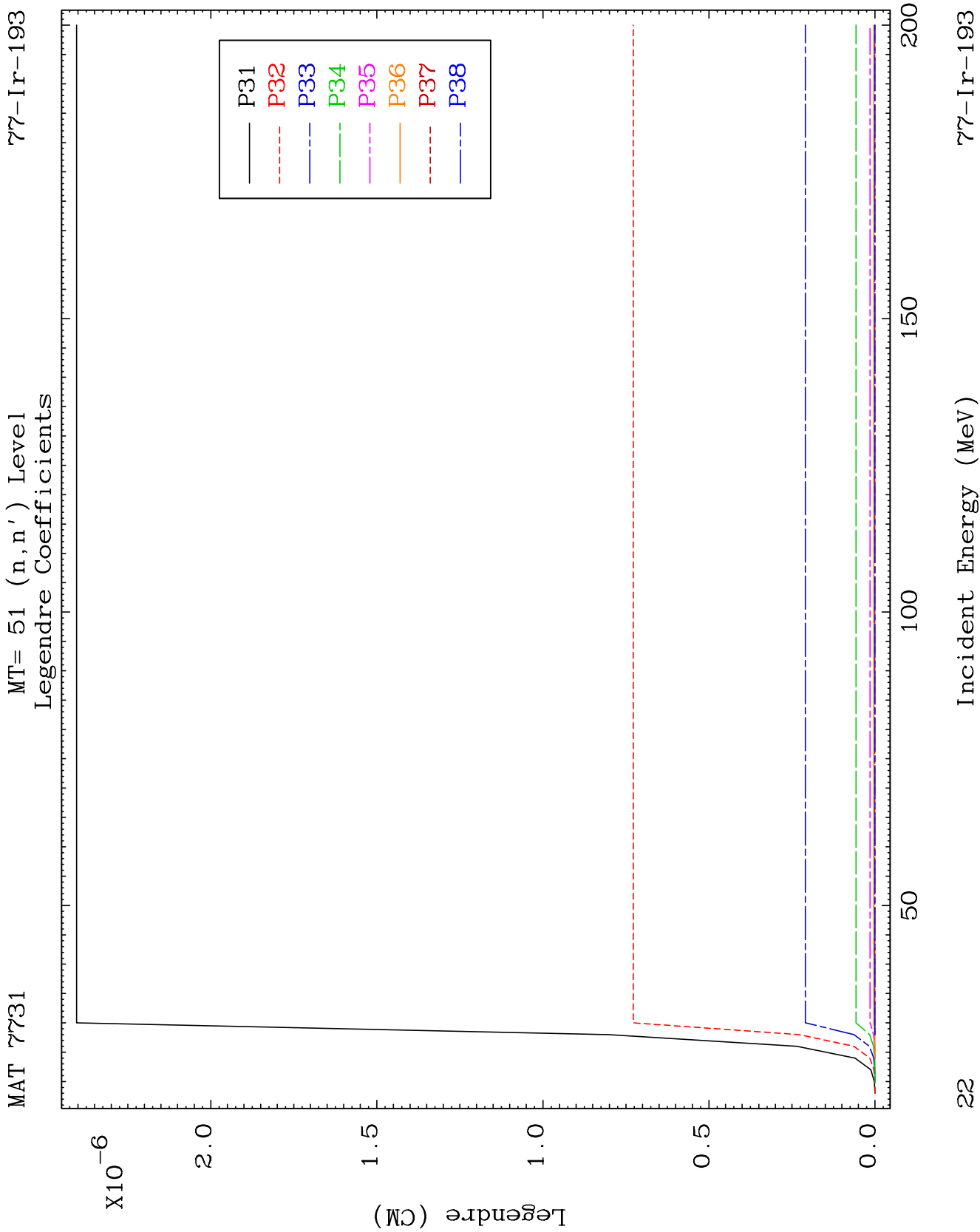
MAT 7731

MT= 51 (n, n') Level
Legendre Coefficients

77-1r-193



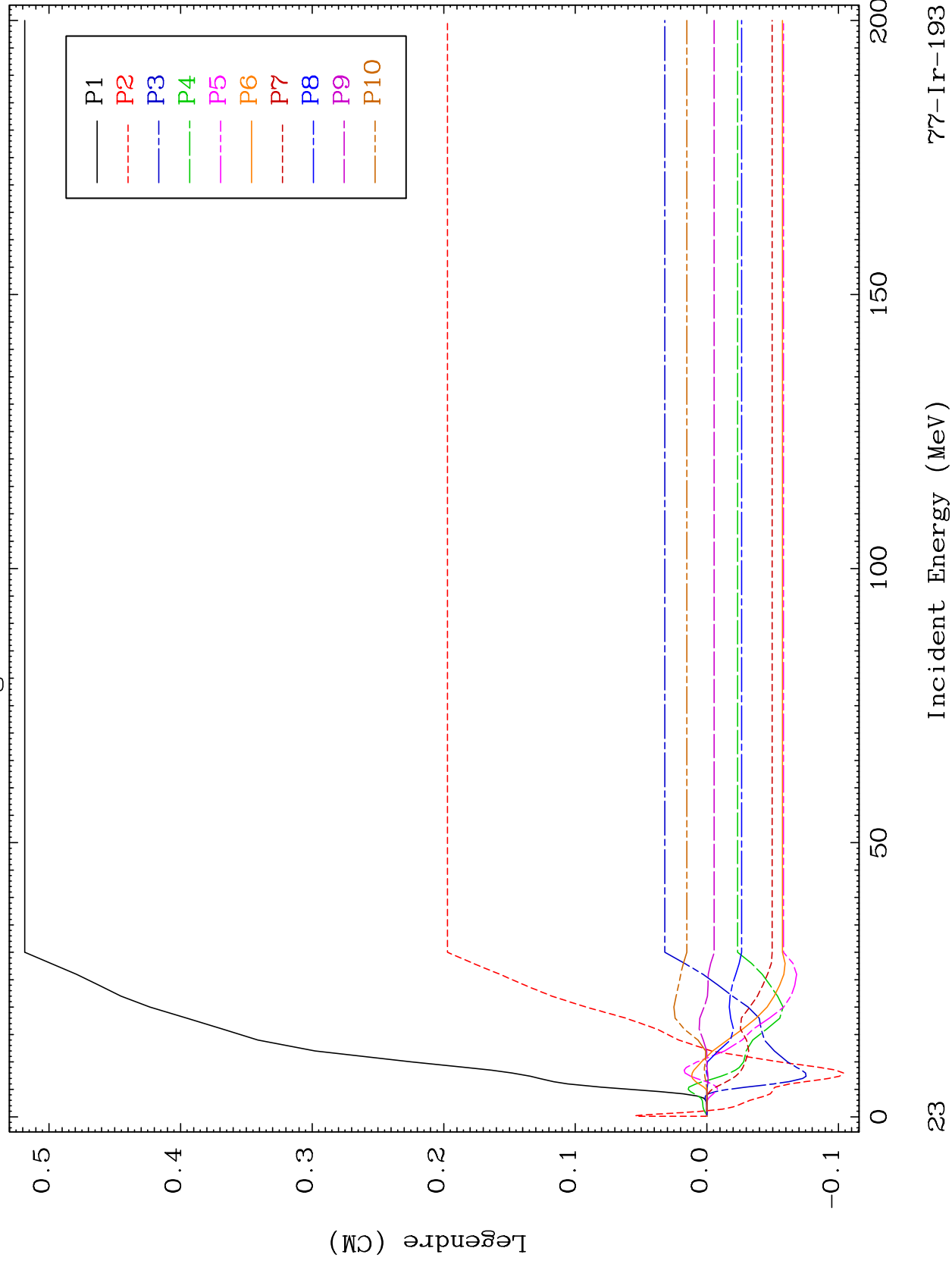


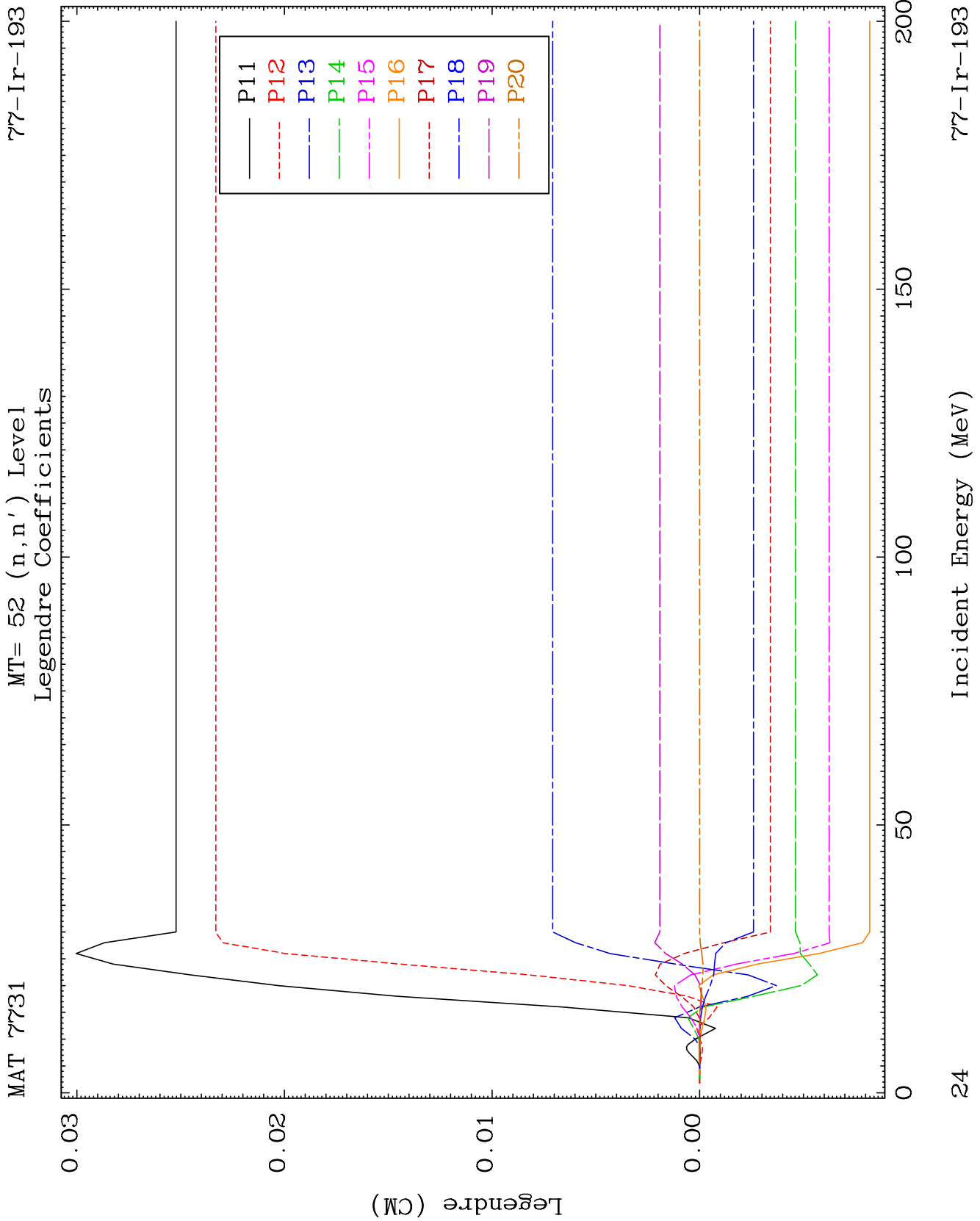


MAT 7731

MT= 52 (n, n') Level
Legendre Coefficients

77-Ir-193

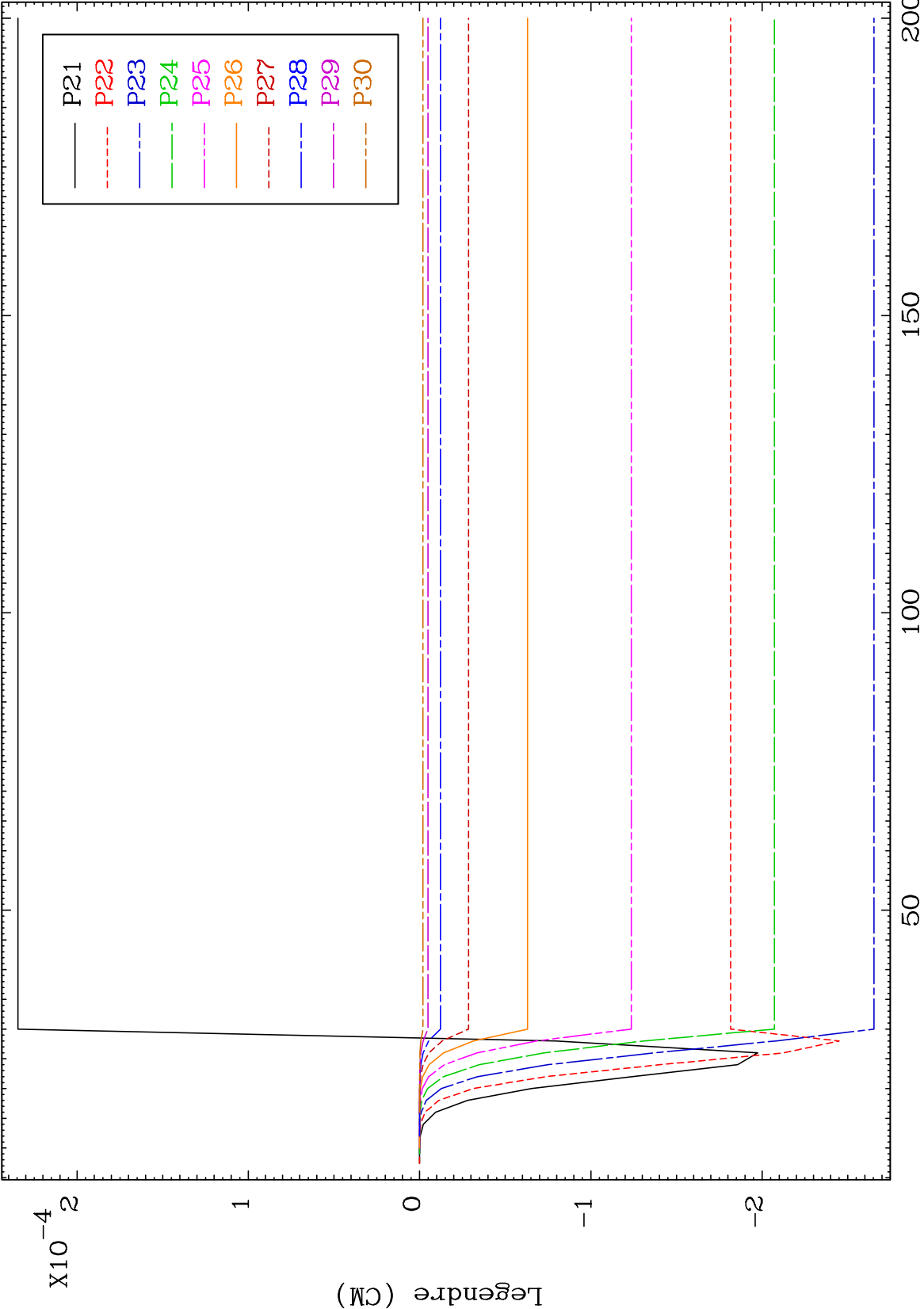




MAT 7731

MT= 52 (n,n') Level
Legendre Coefficients

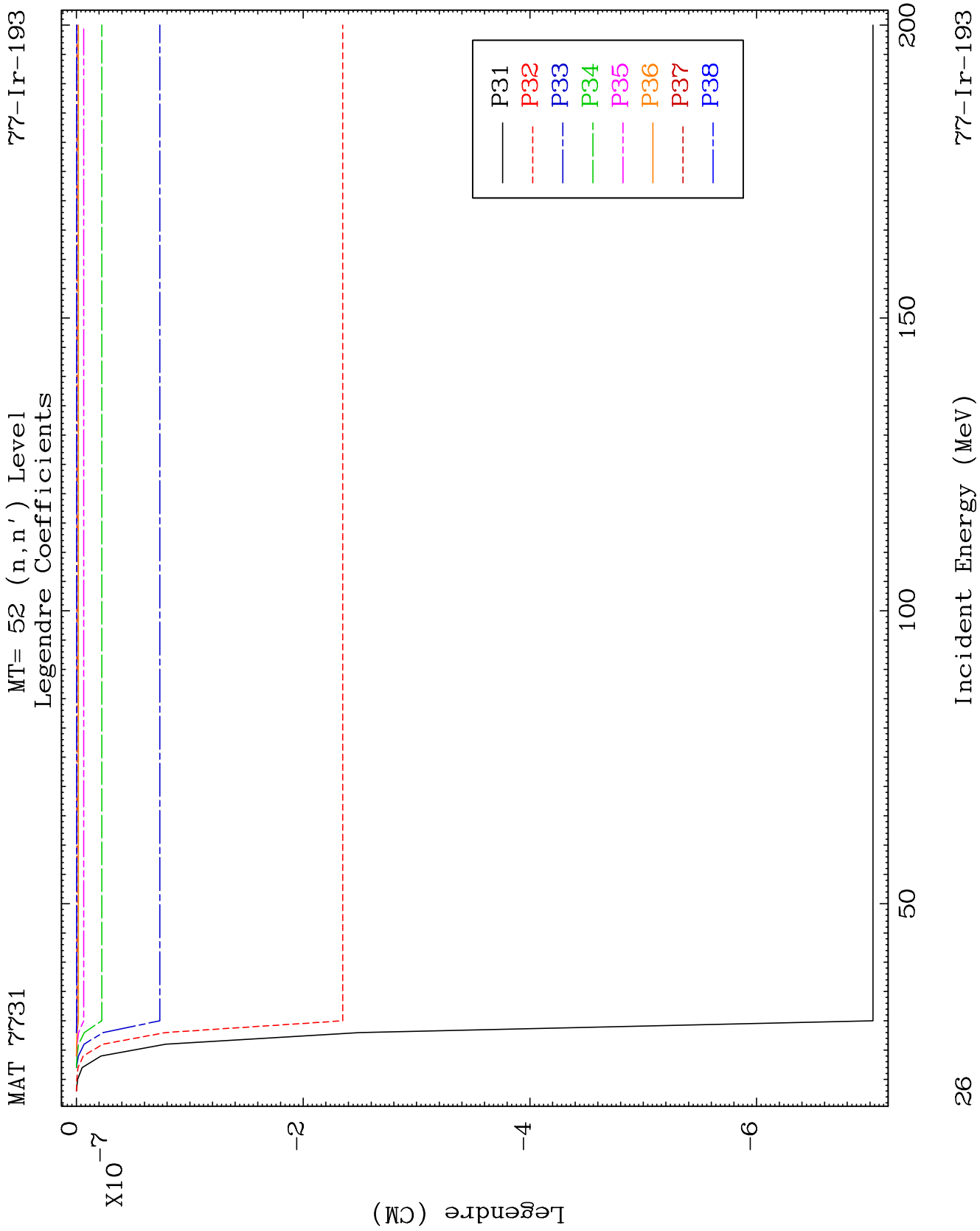
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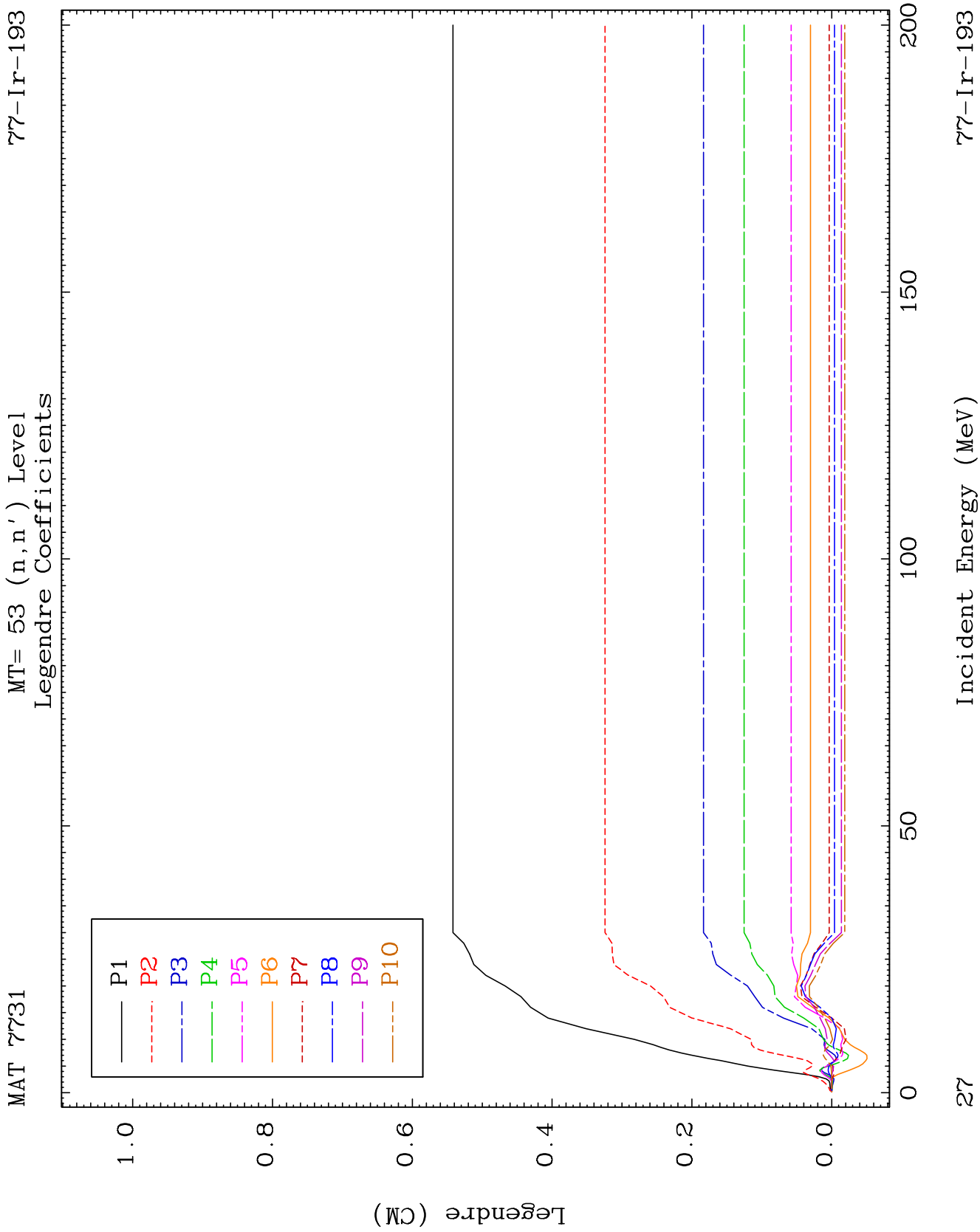


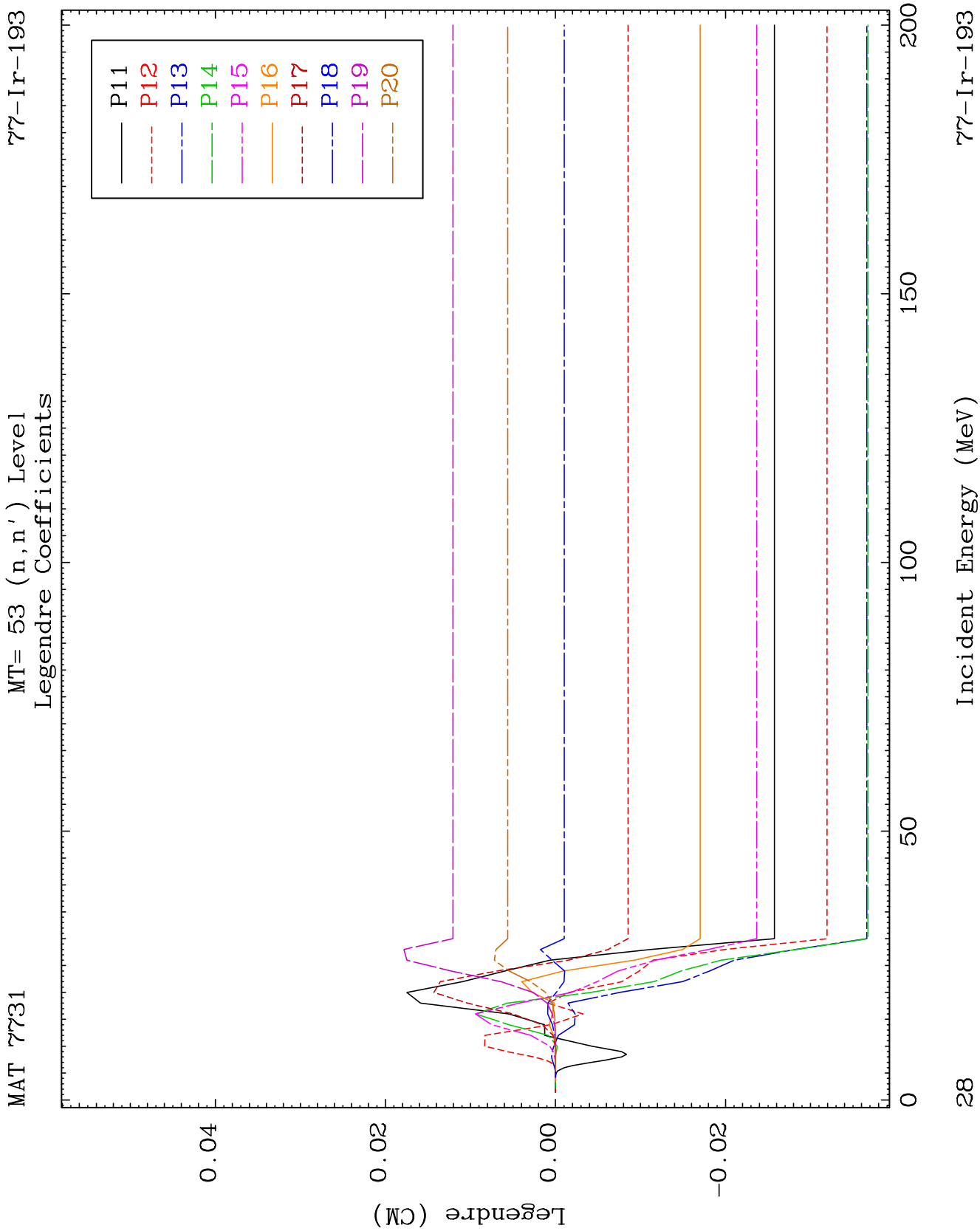
25

Incident Energy (MeV)

77-Ir-193



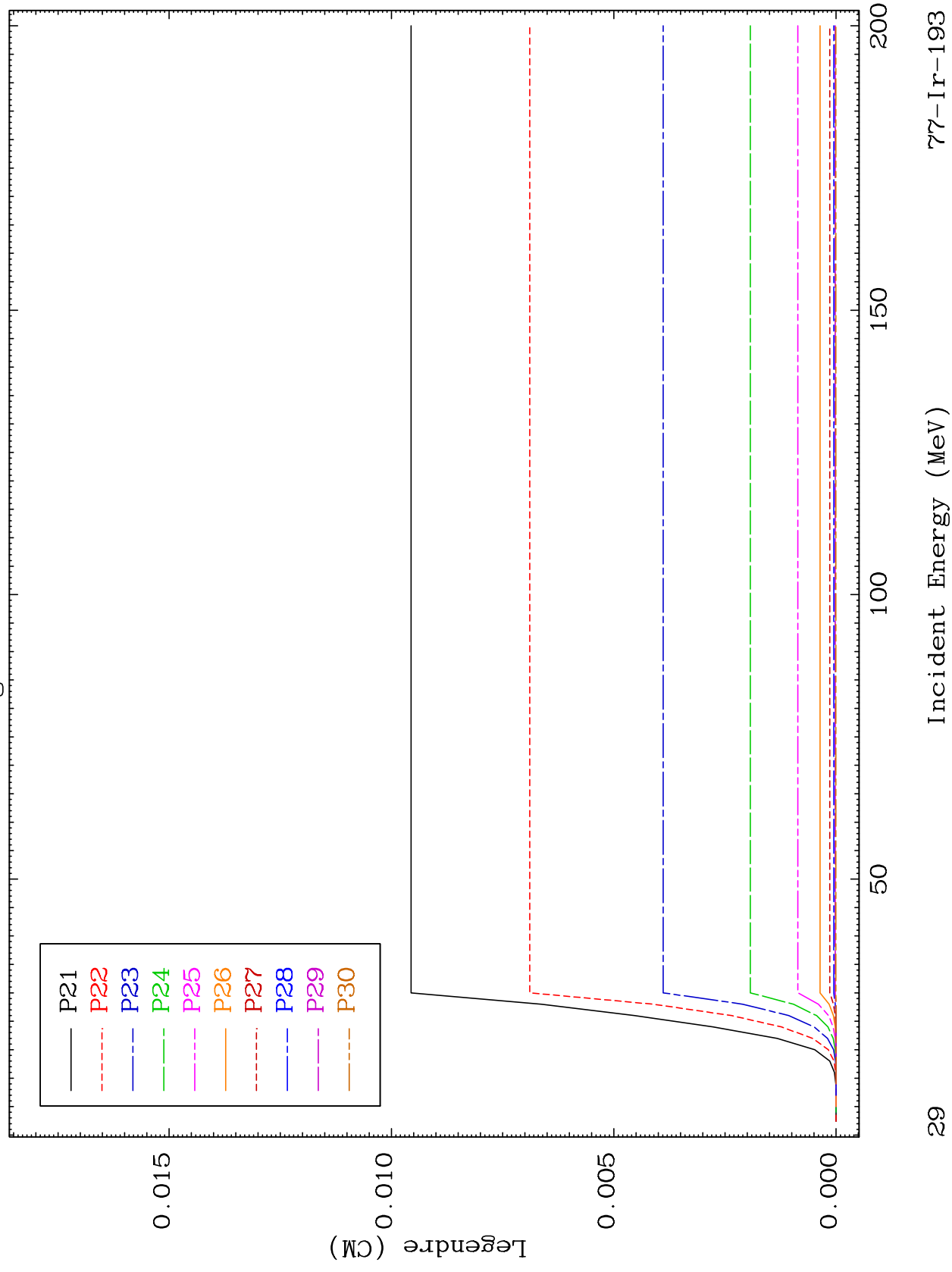


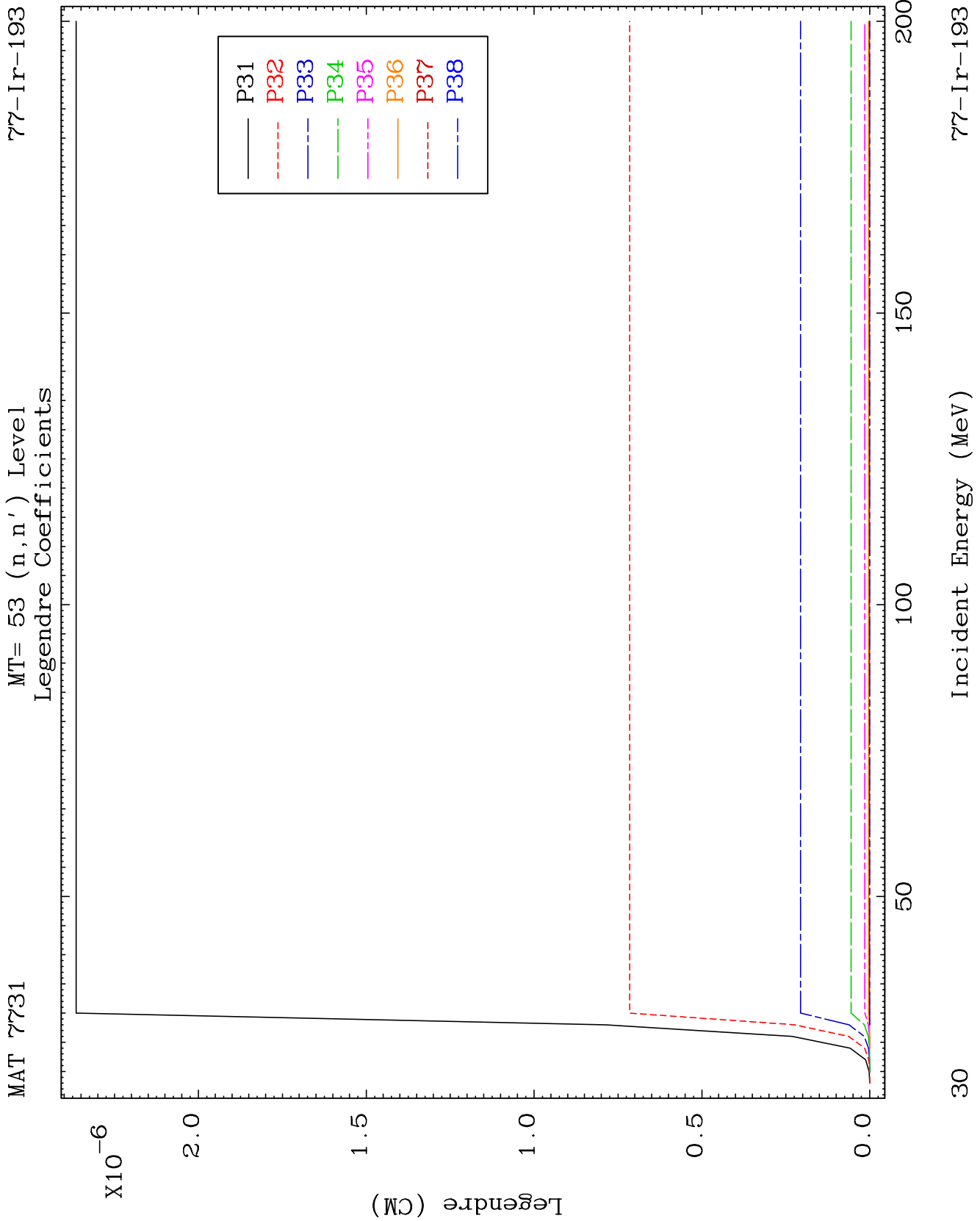


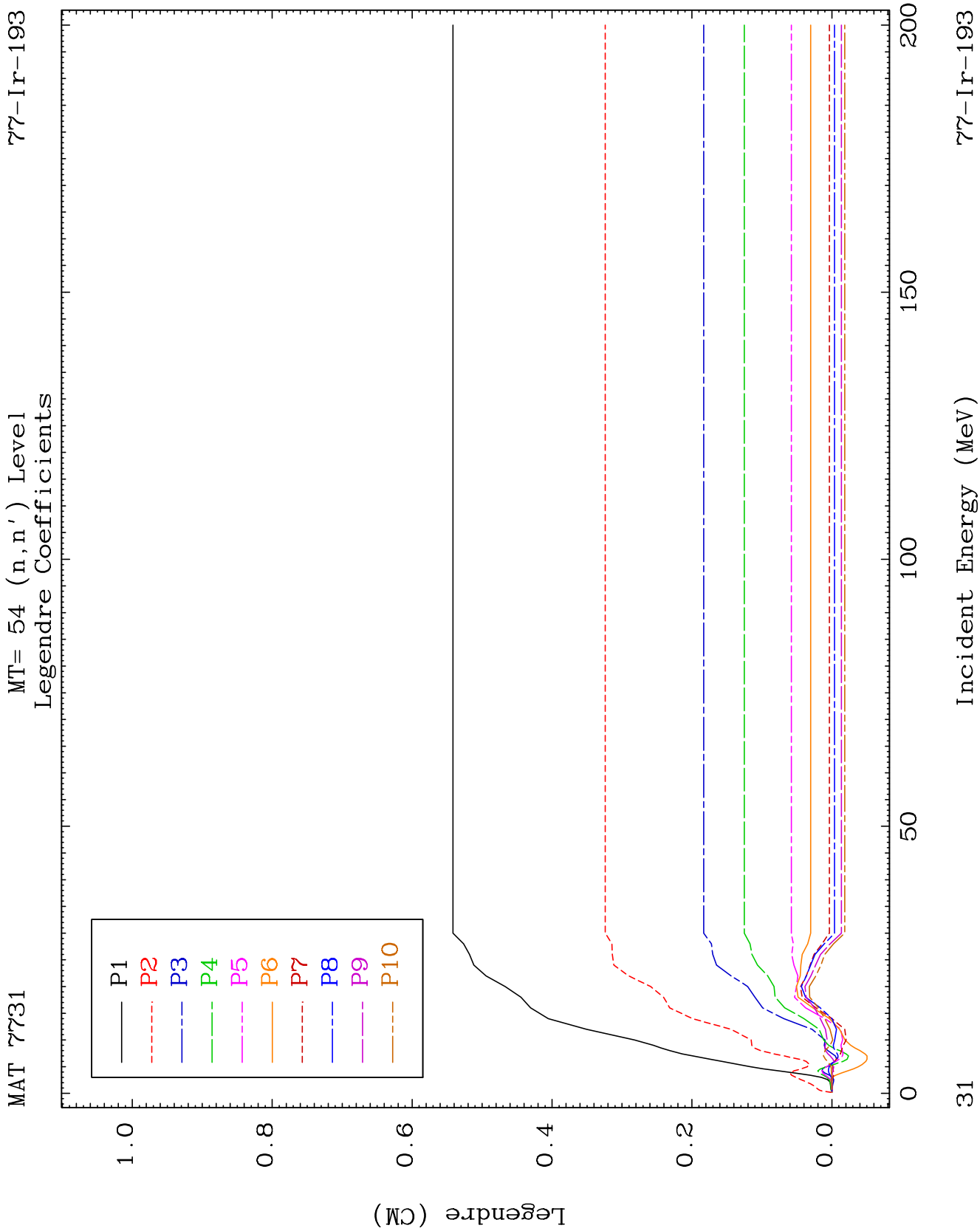
MAT 7731

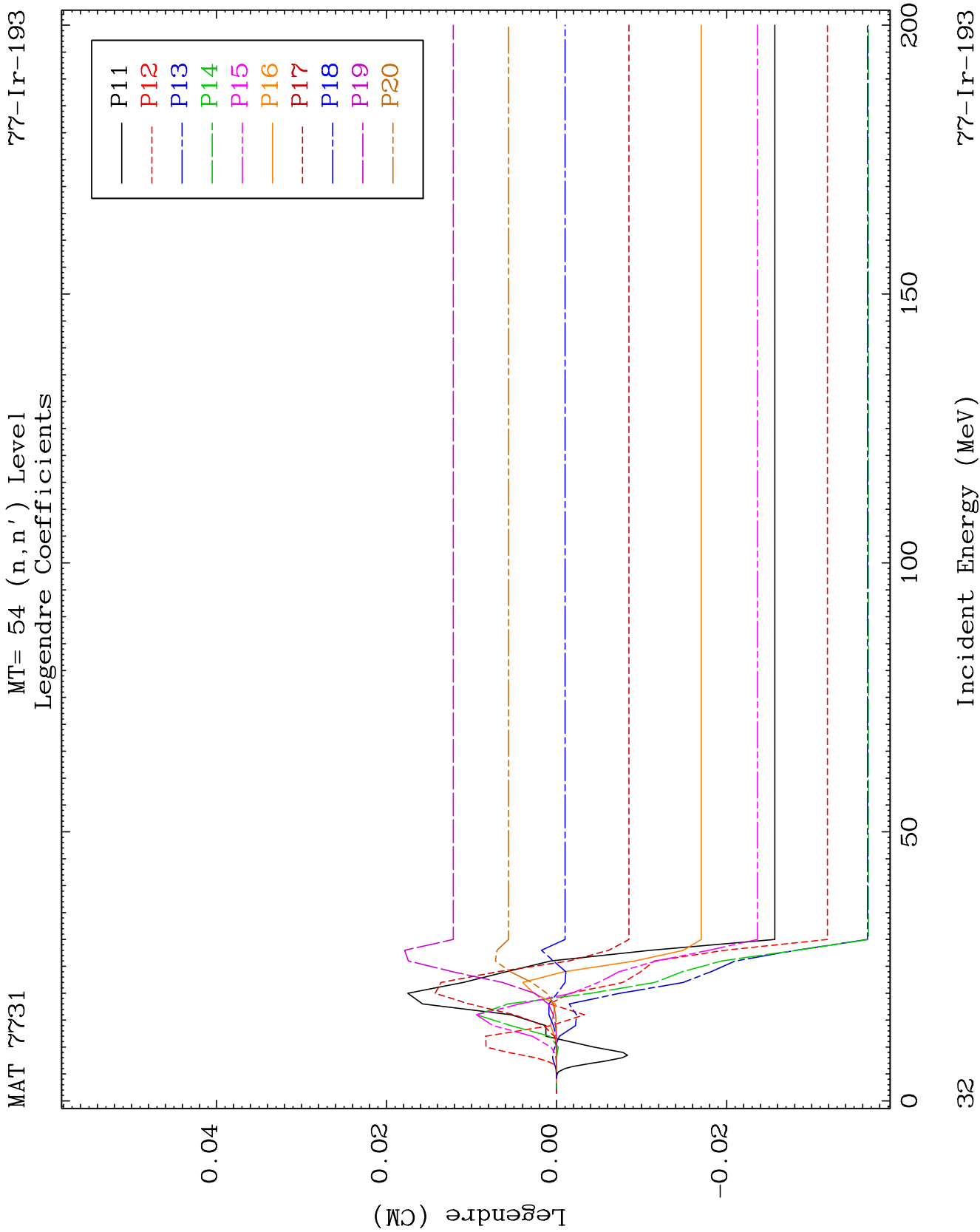
MT= 53 (n, n') Level
Legendre Coefficients

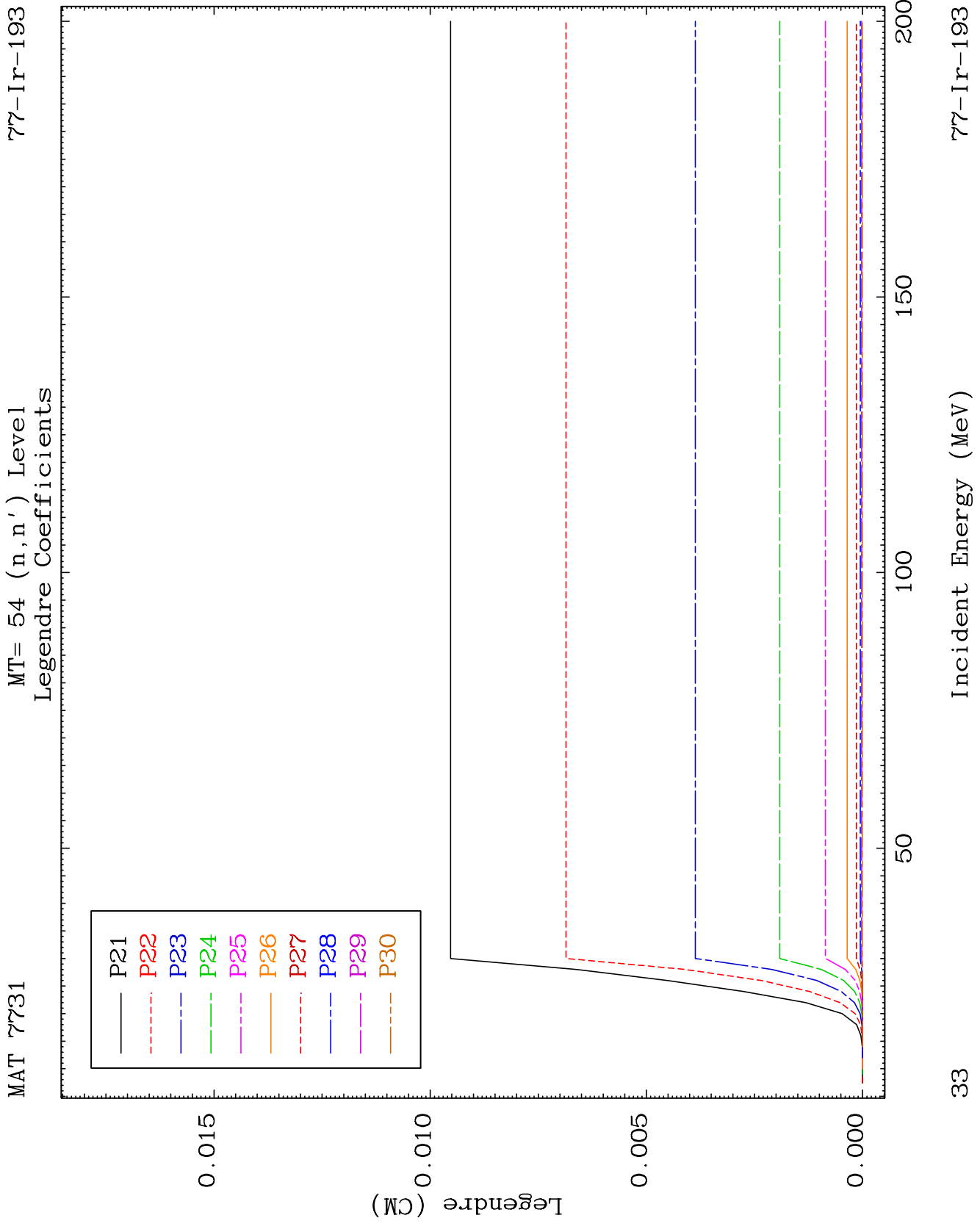
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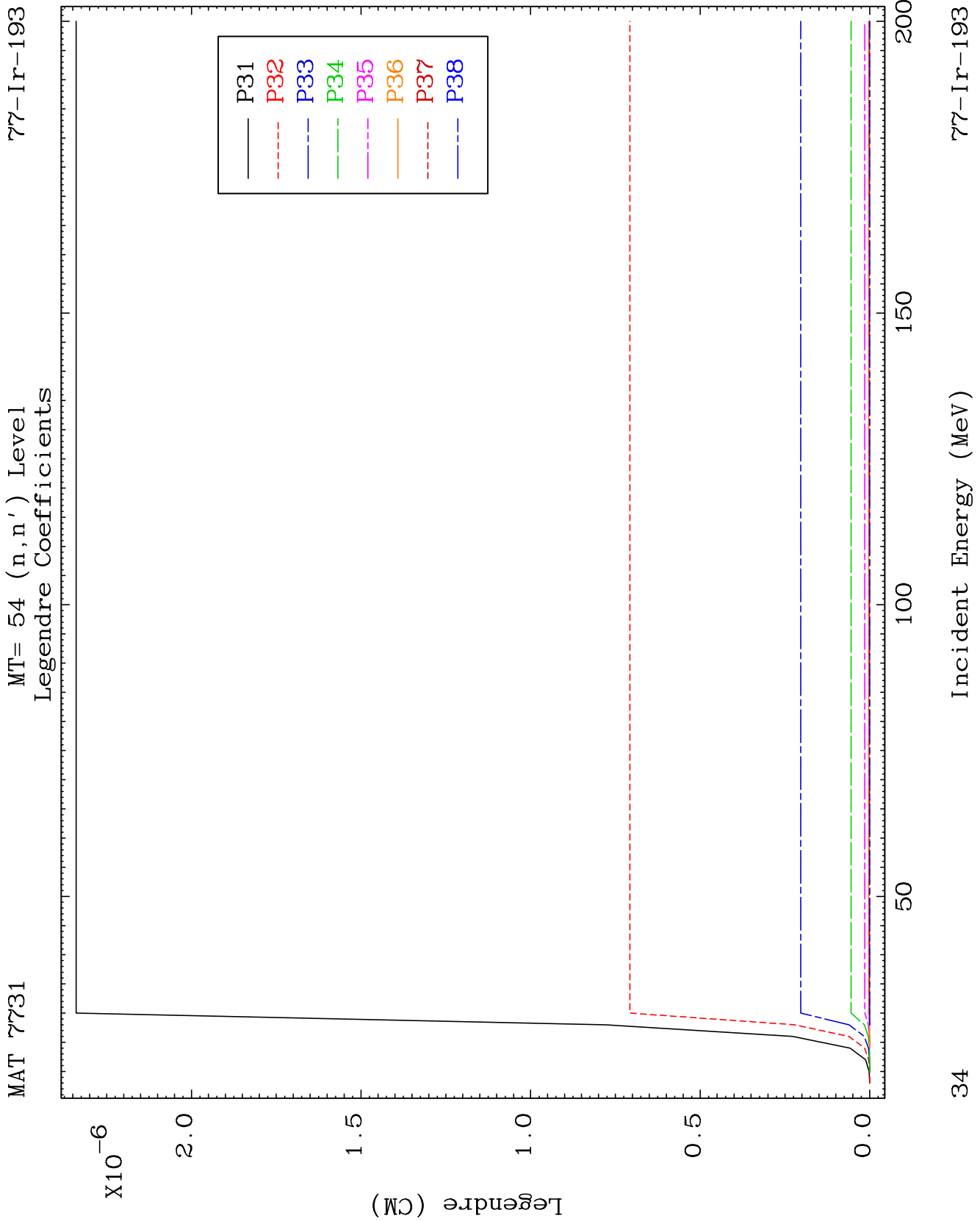








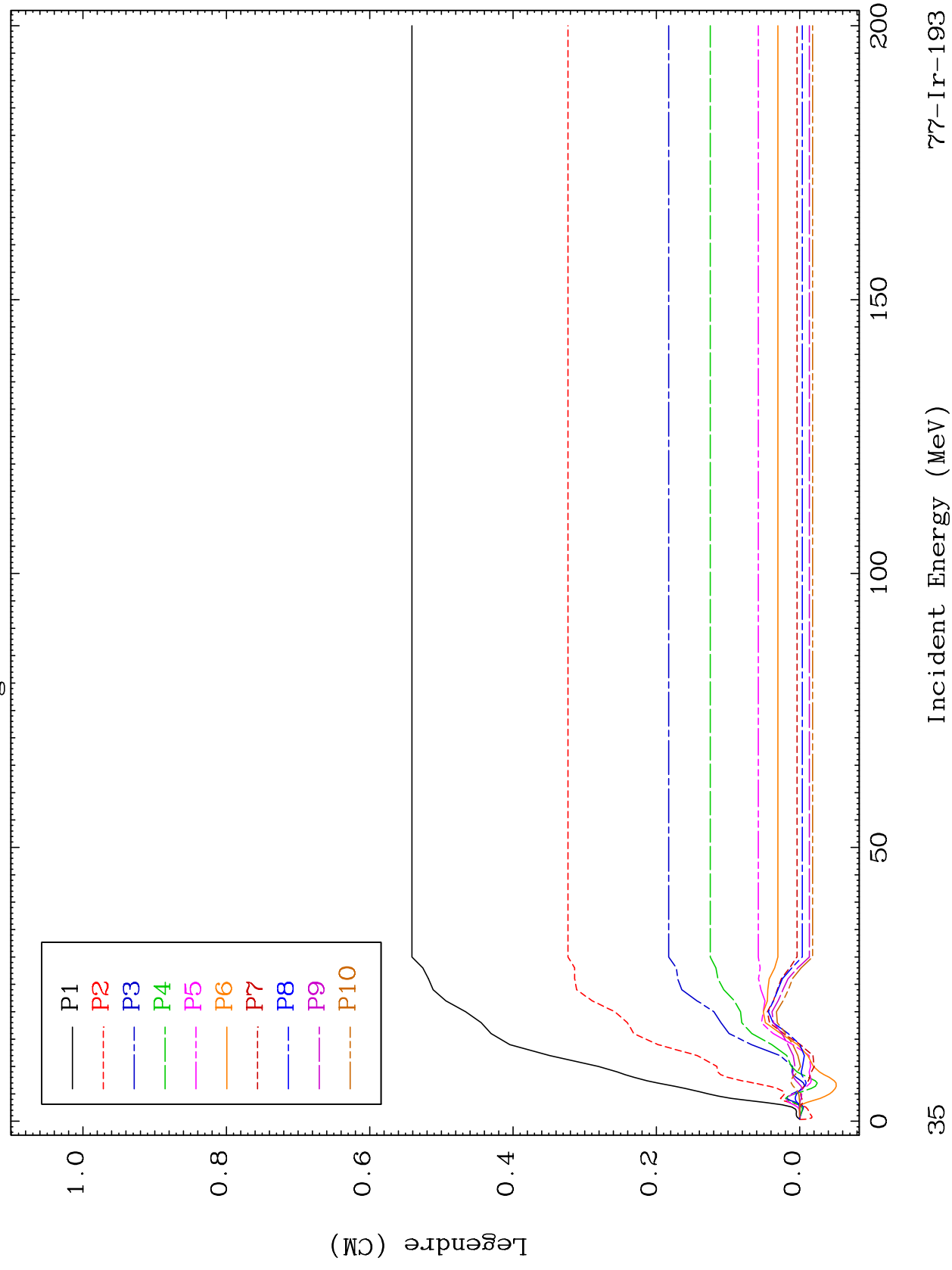


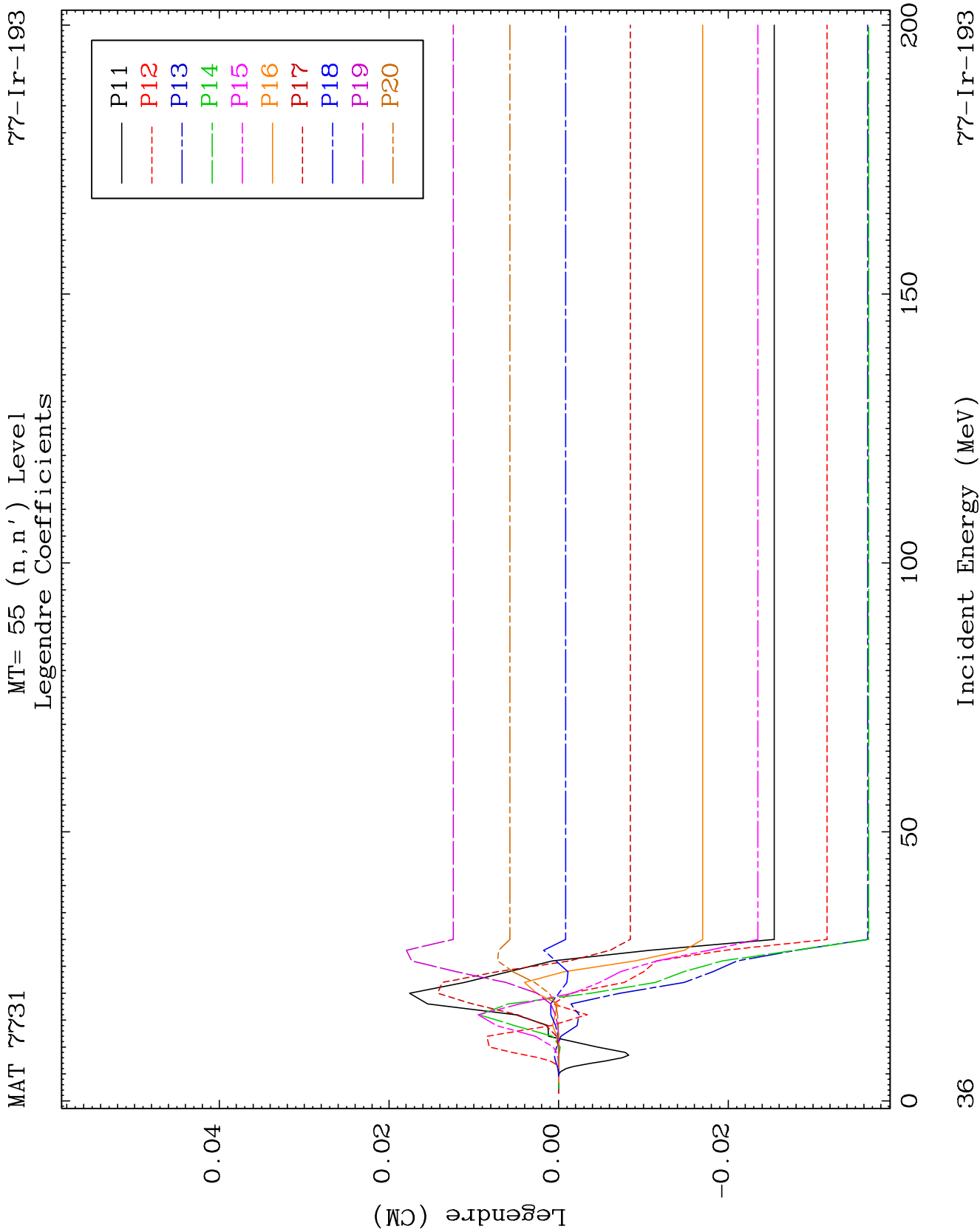


MAT 7731

MT= 55 (n, n') Level
Legendre Coefficients

77-Ir-193

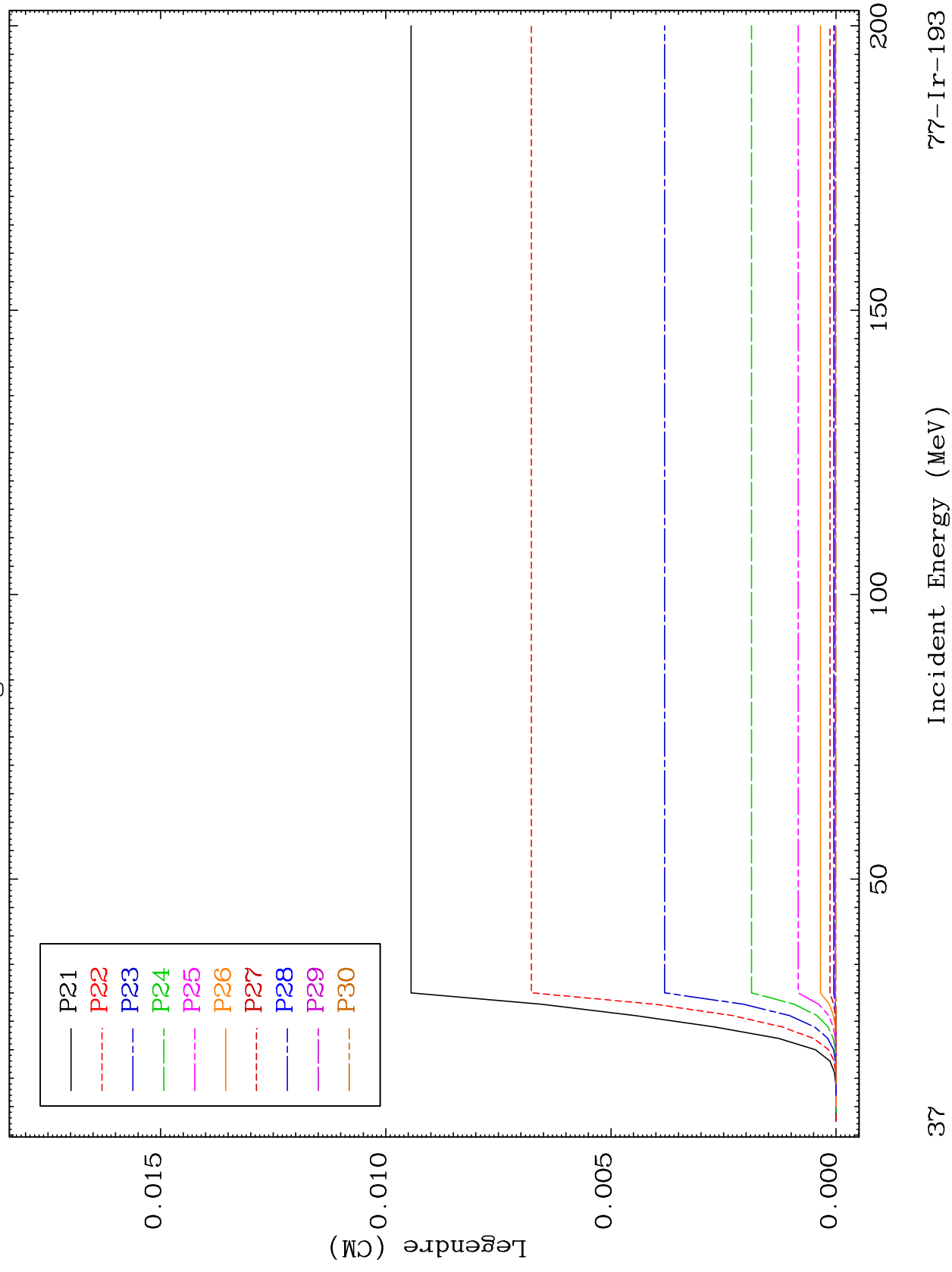


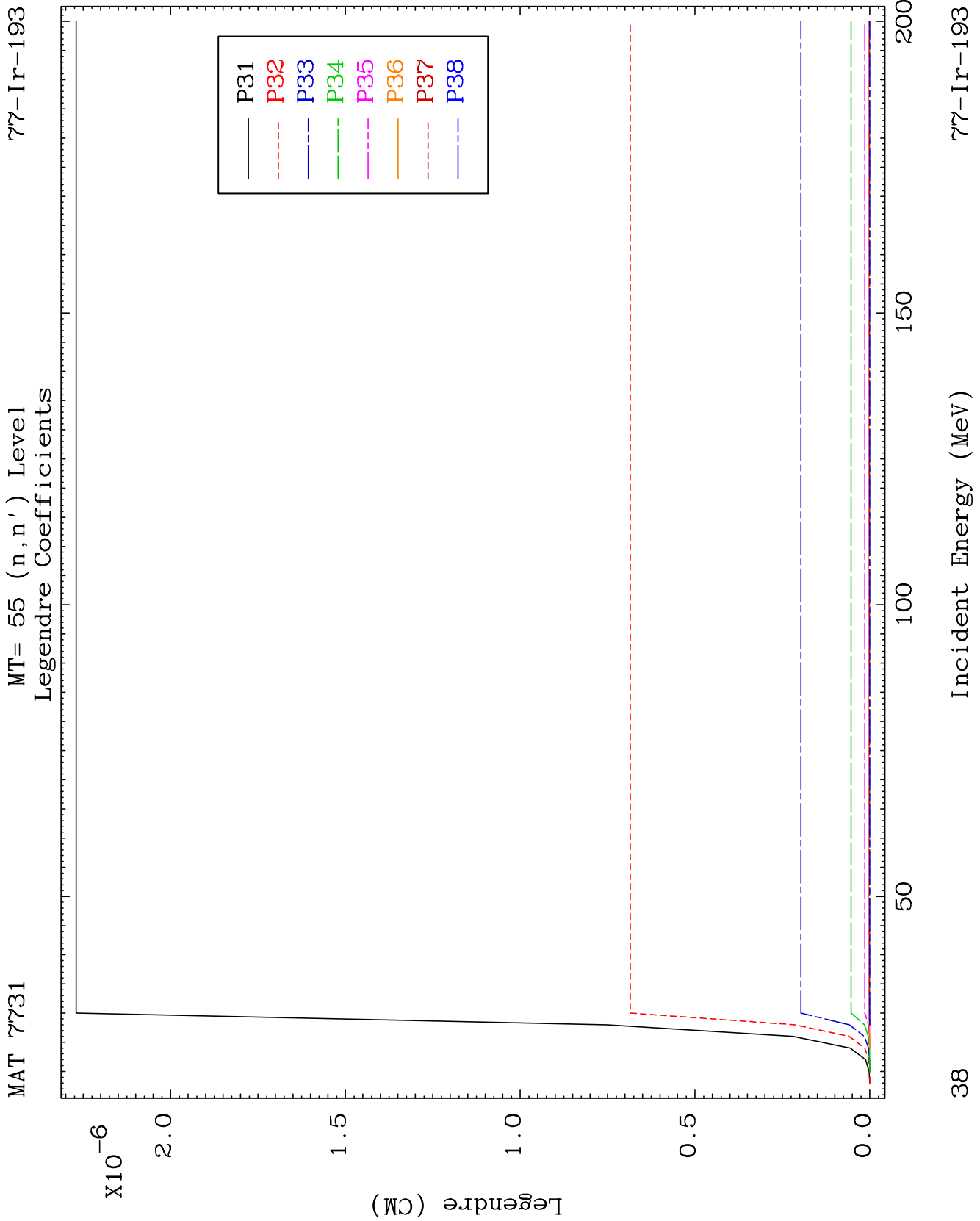


MAT 7731

MT= 55 (n, n') Level
Legendre Coefficients

77-Ir-193

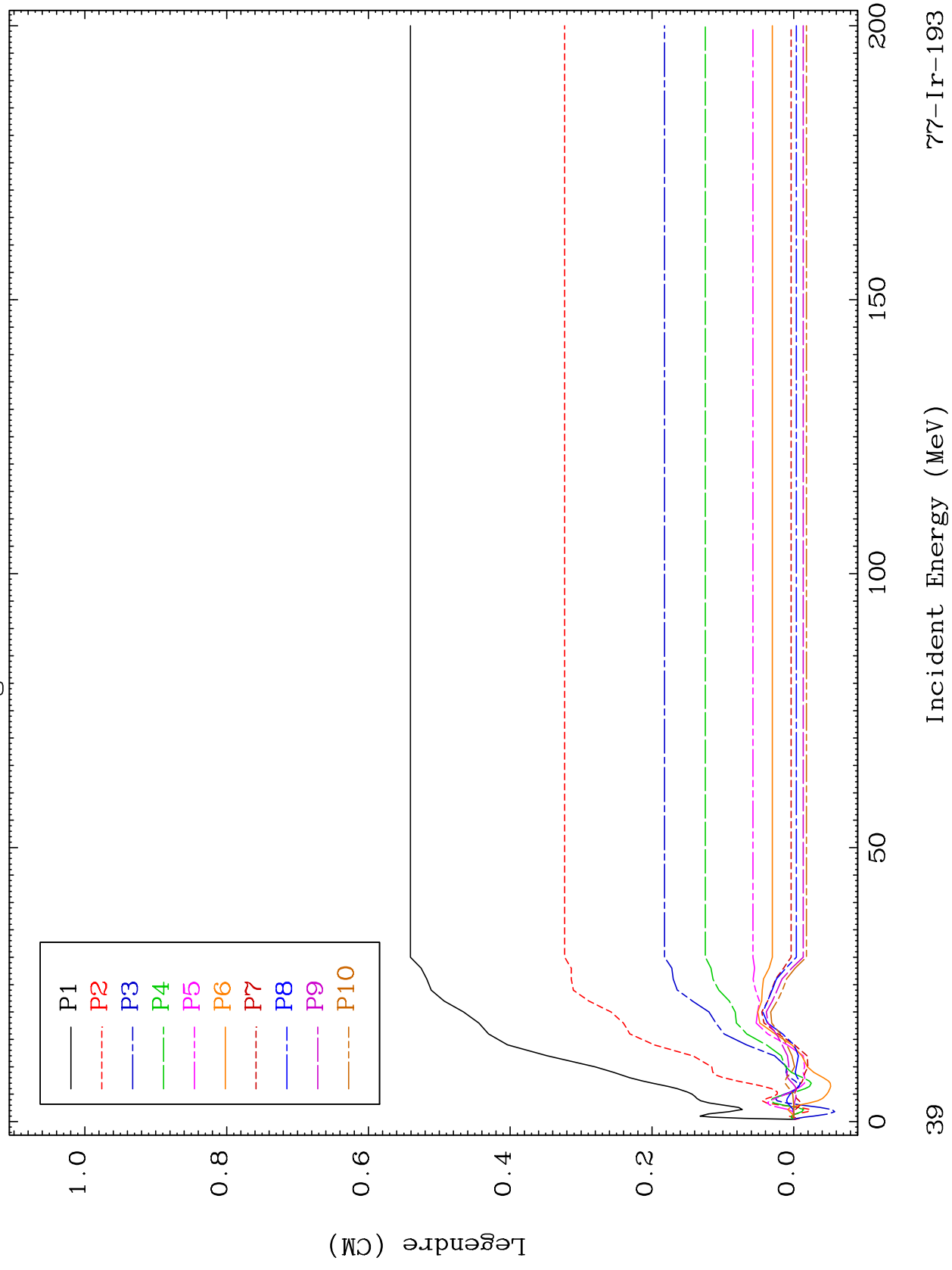




MAT 7731

MT= 56 (n, n') Level
Legendre Coefficients

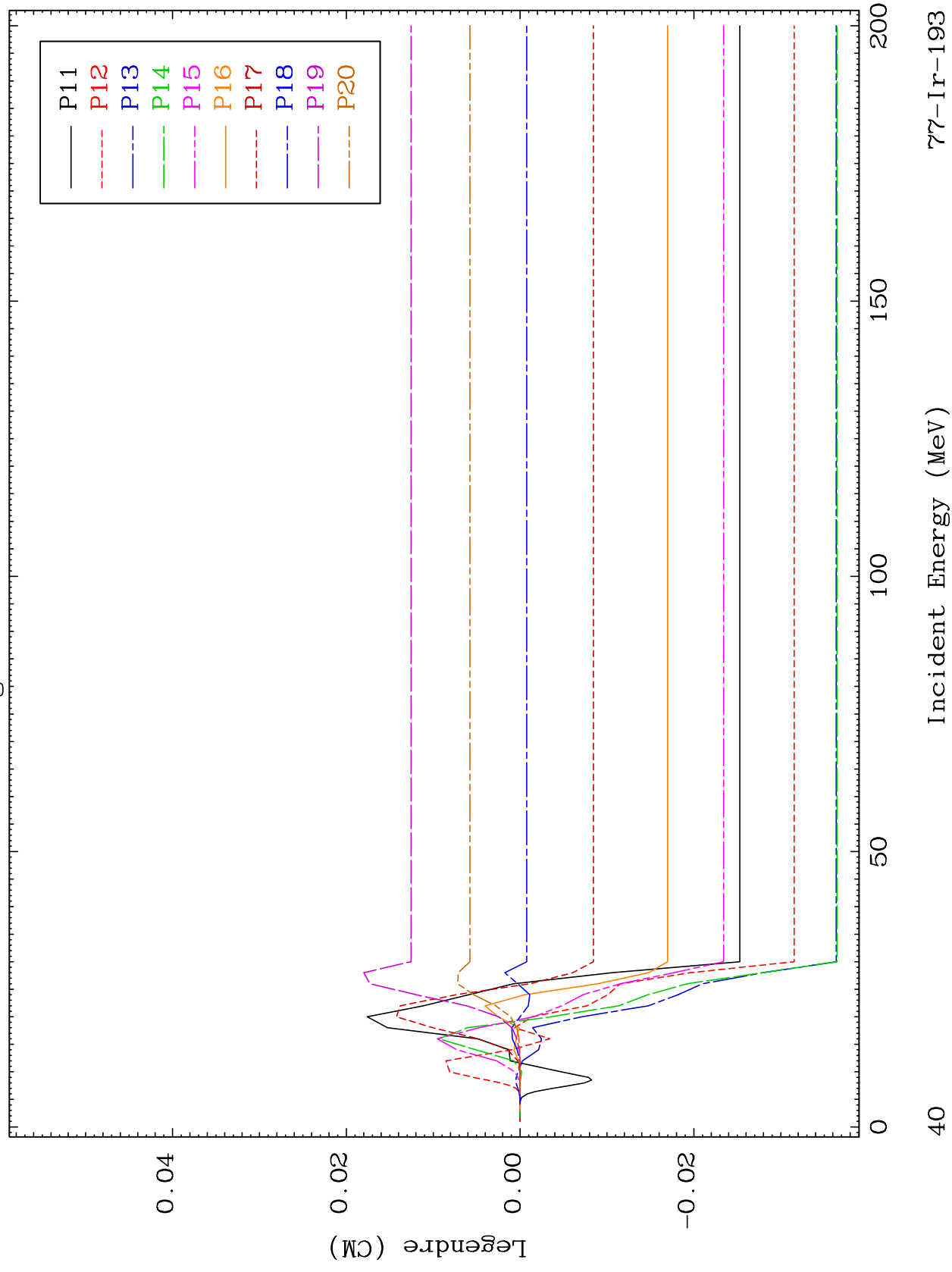
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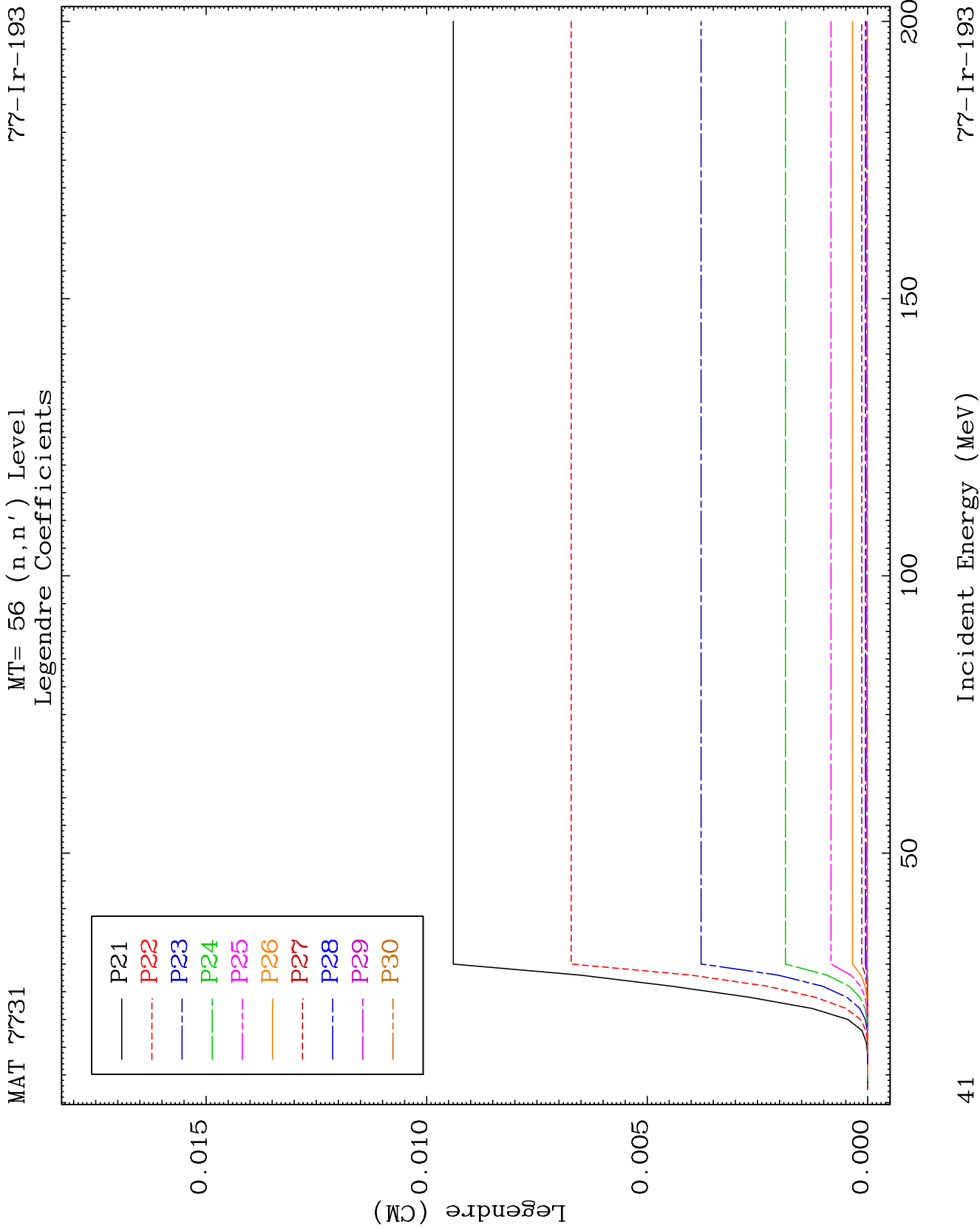


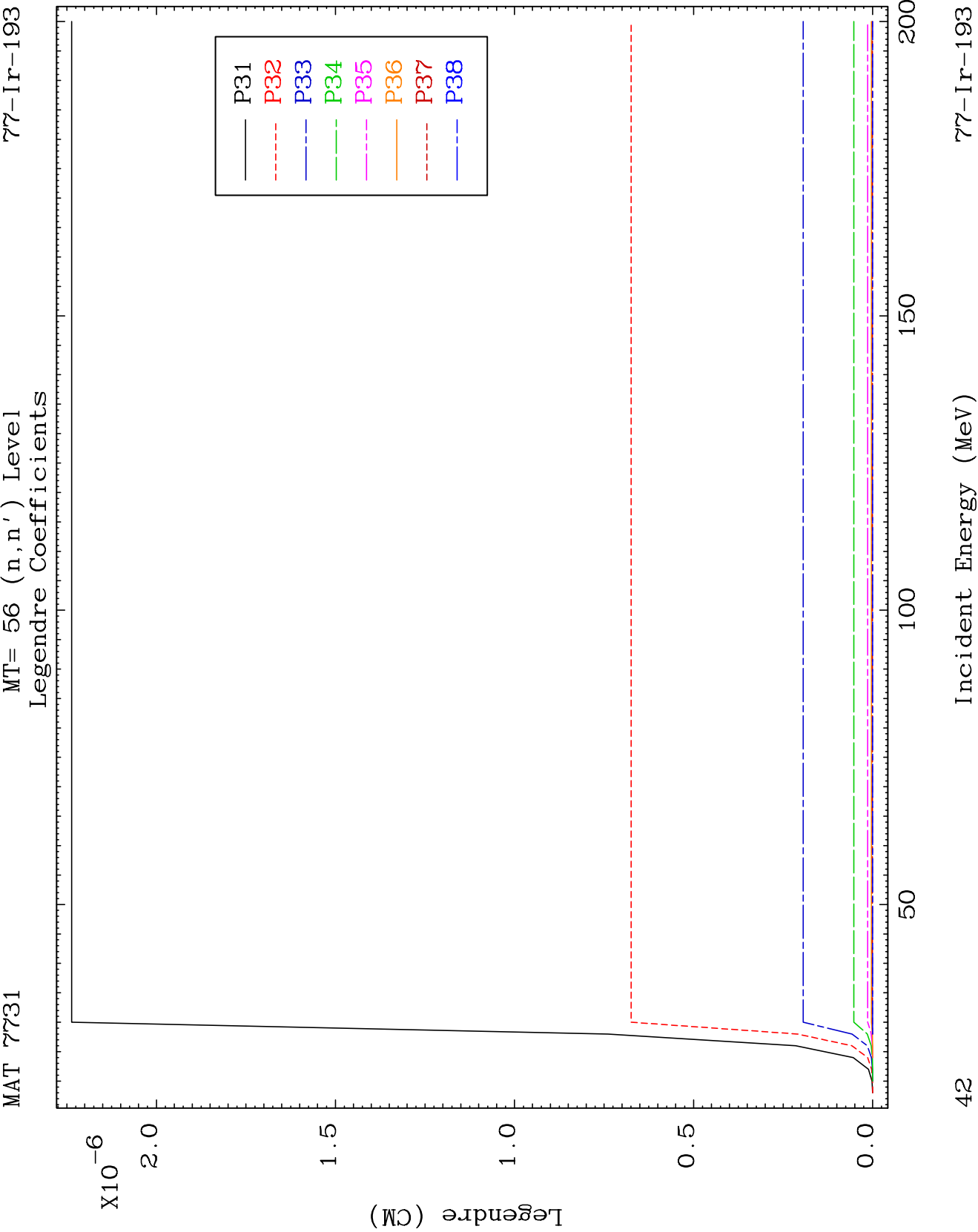
MAT 7731

MT= 56 (n, n') Level
Legendre Coefficients

77-Ir-193



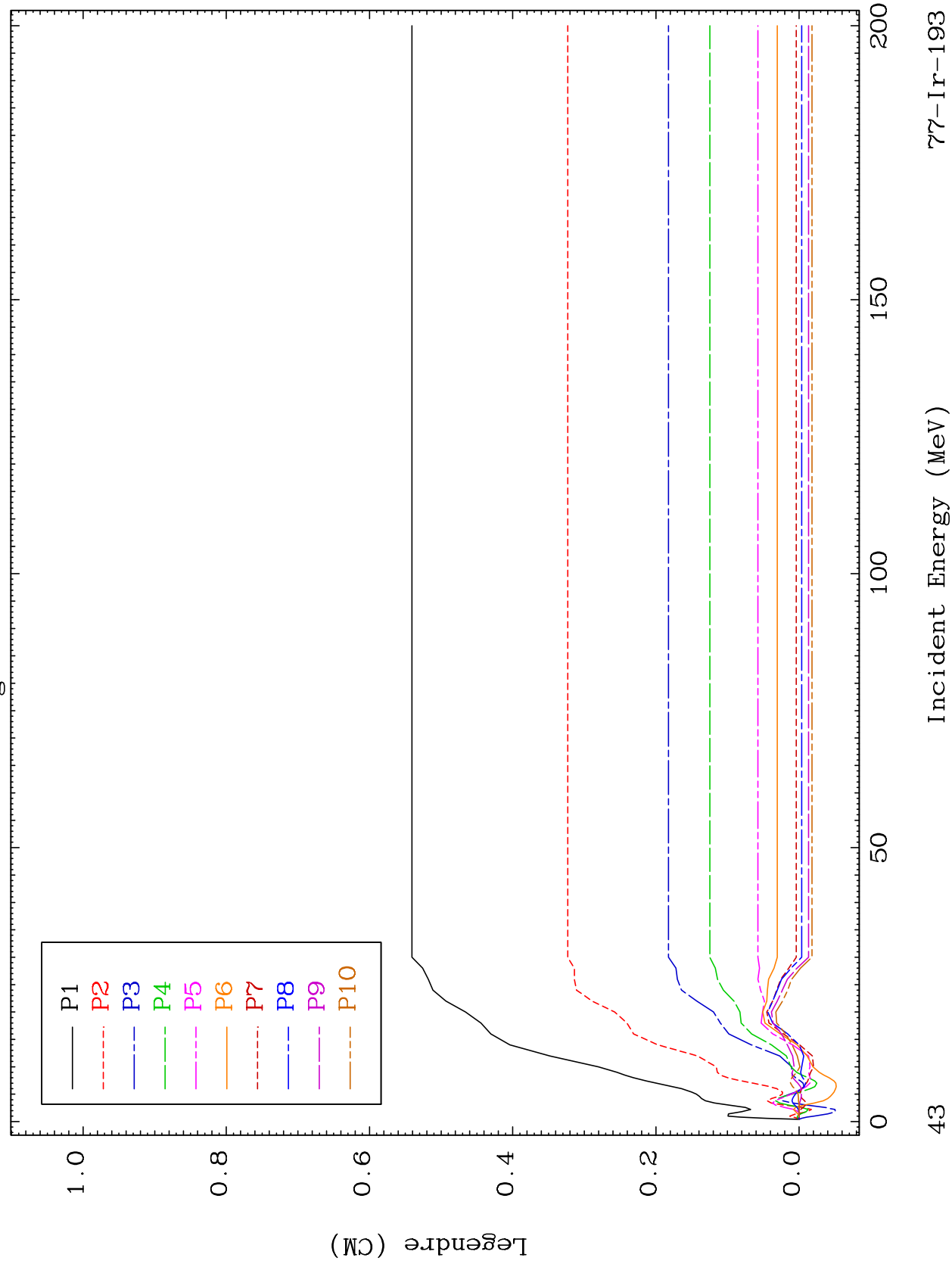


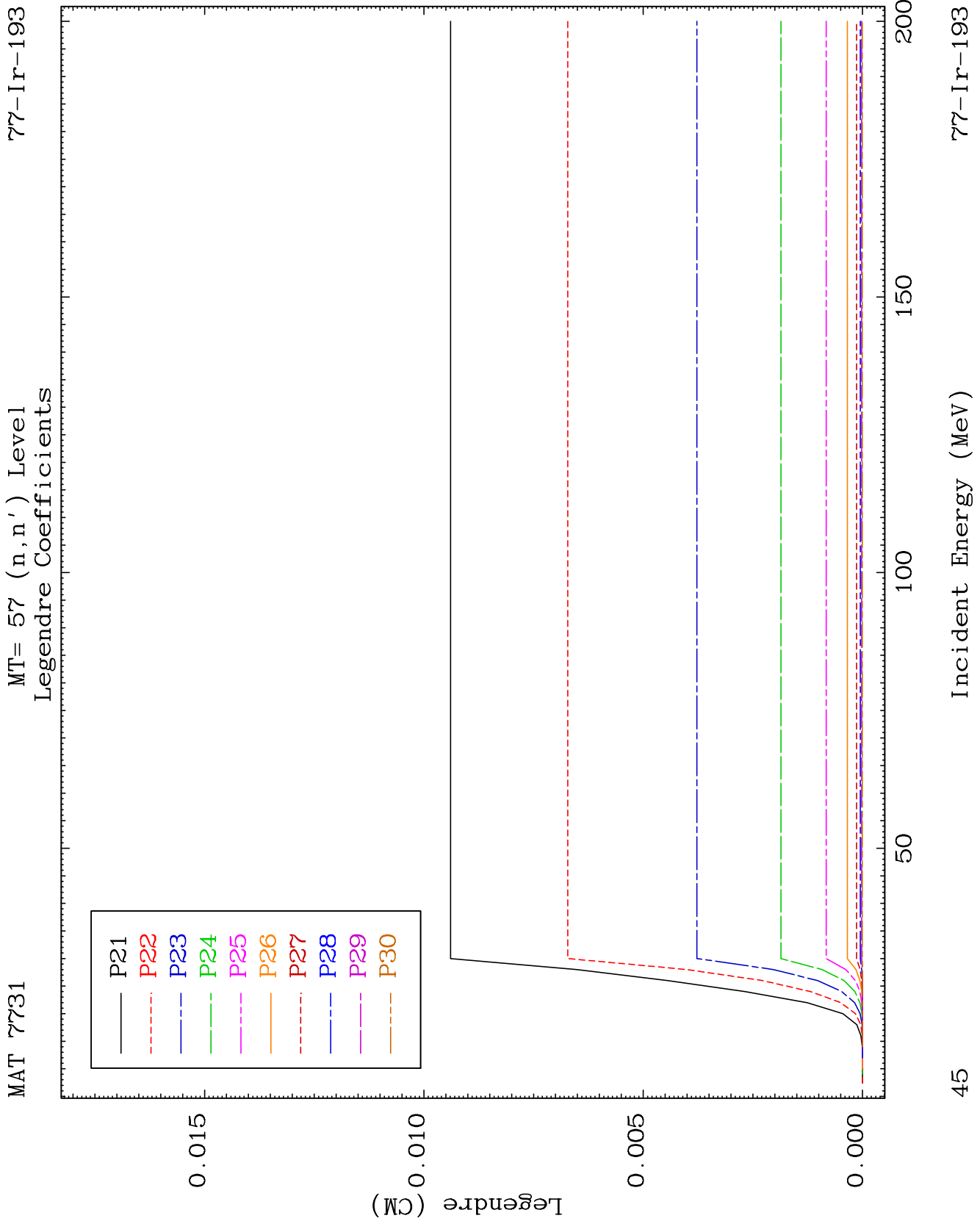


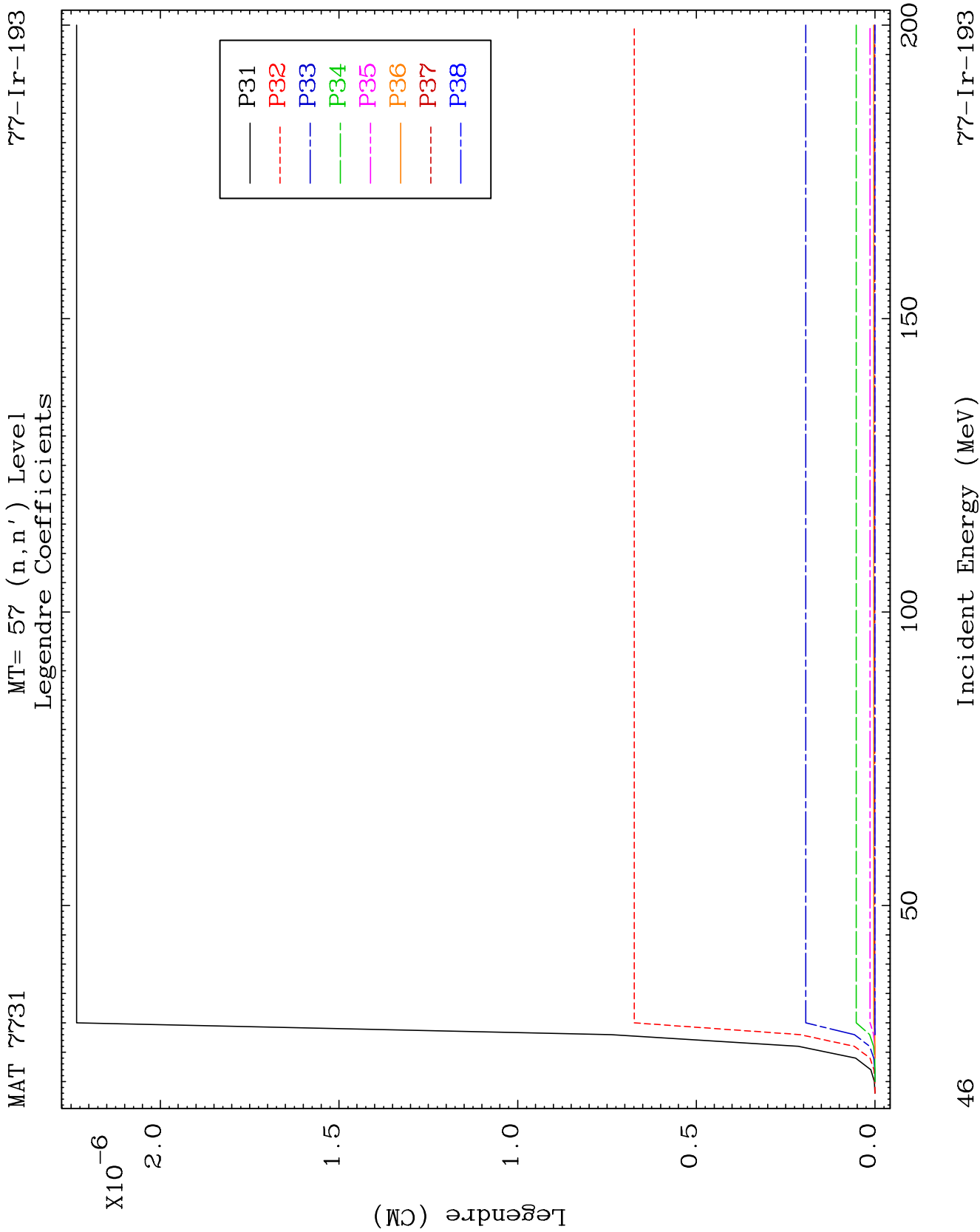
MAT 7731

MT= 57 (n, n') Level
Legendre Coefficients

77-Ir-193



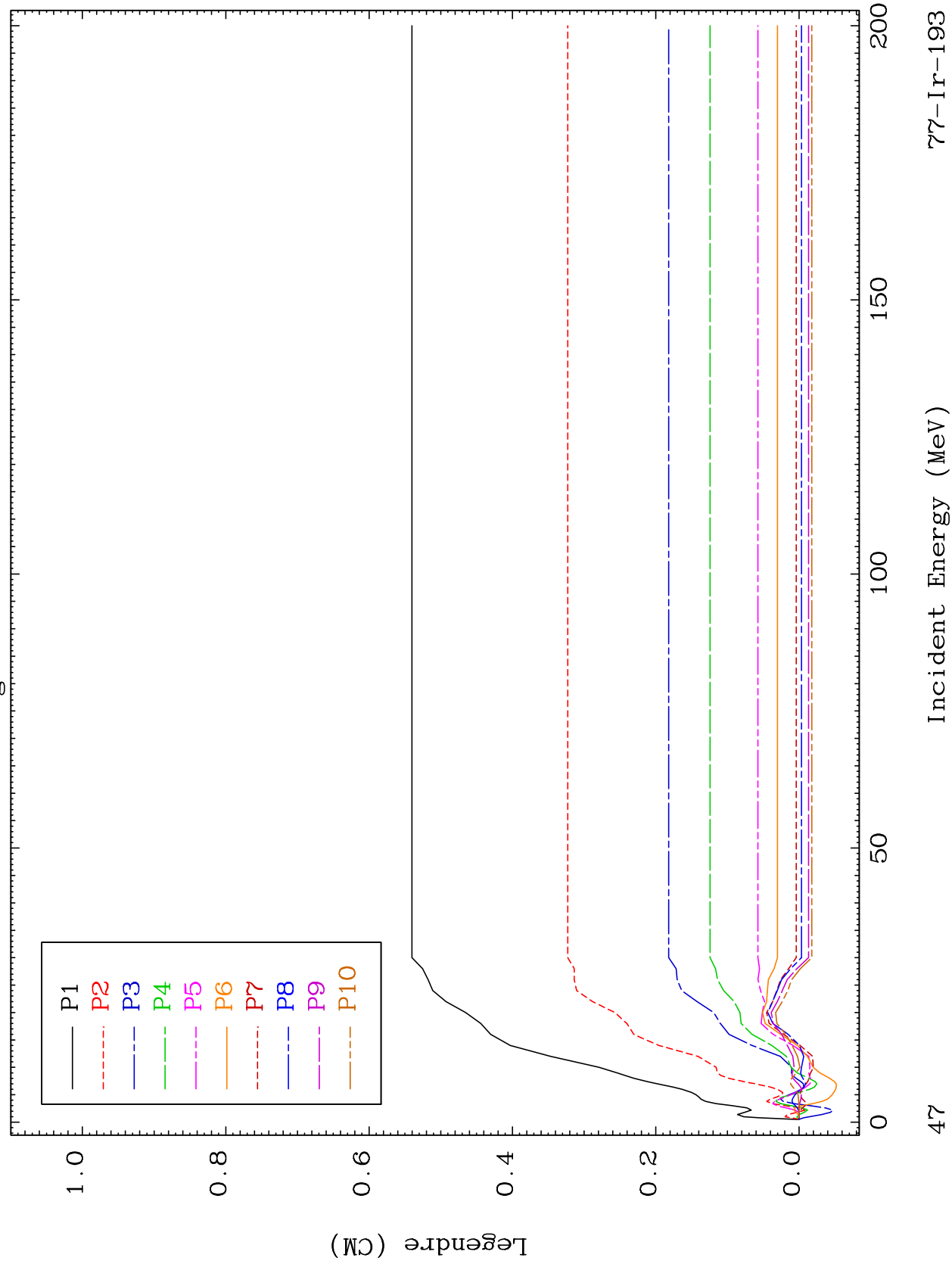




MAT 7731

MT= 58 (n, n') Level
Legendre Coefficients

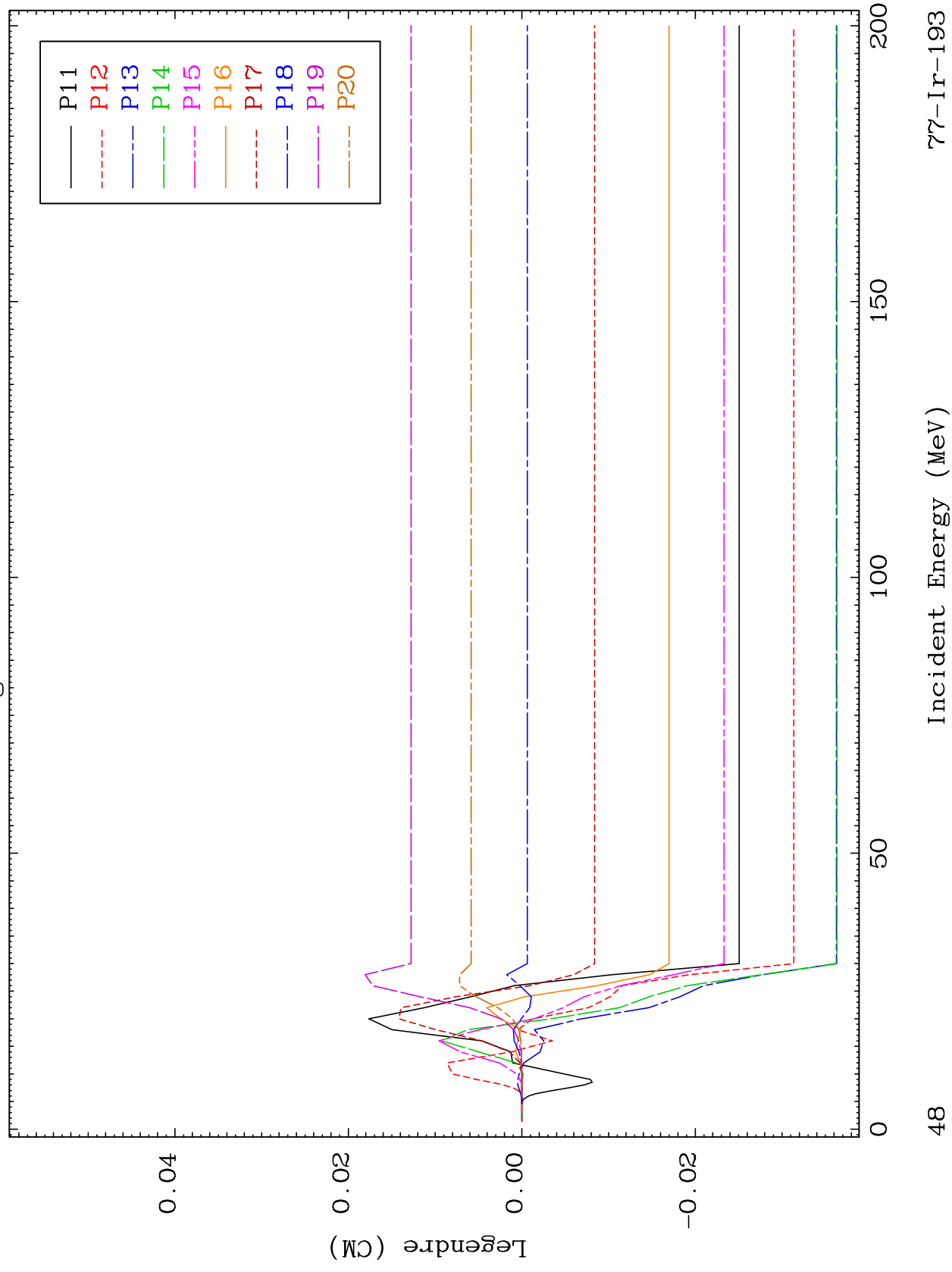
77-Ir-193



MAT 7731

MT= 58 (n, n') Level
Legendre Coefficients

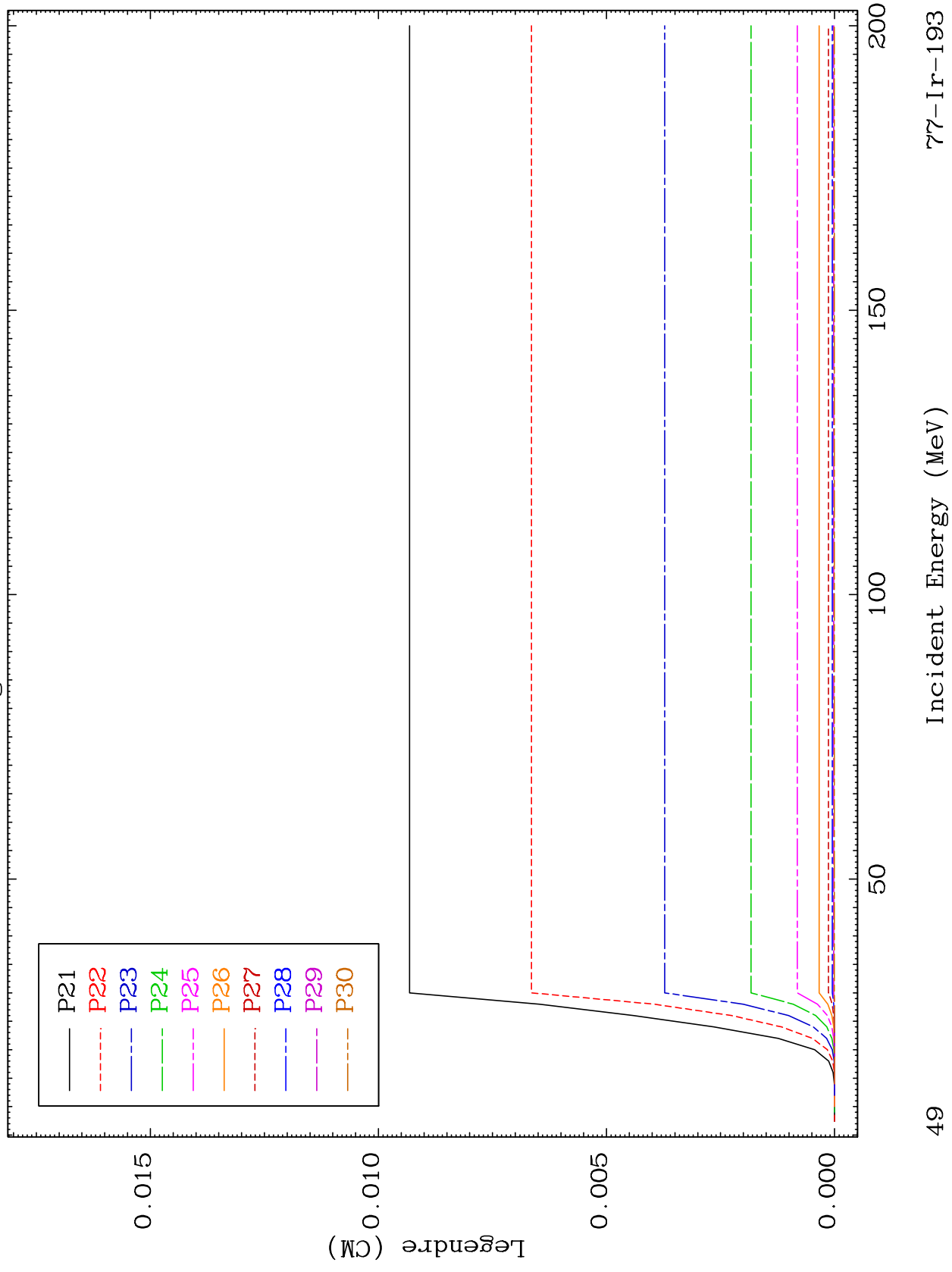
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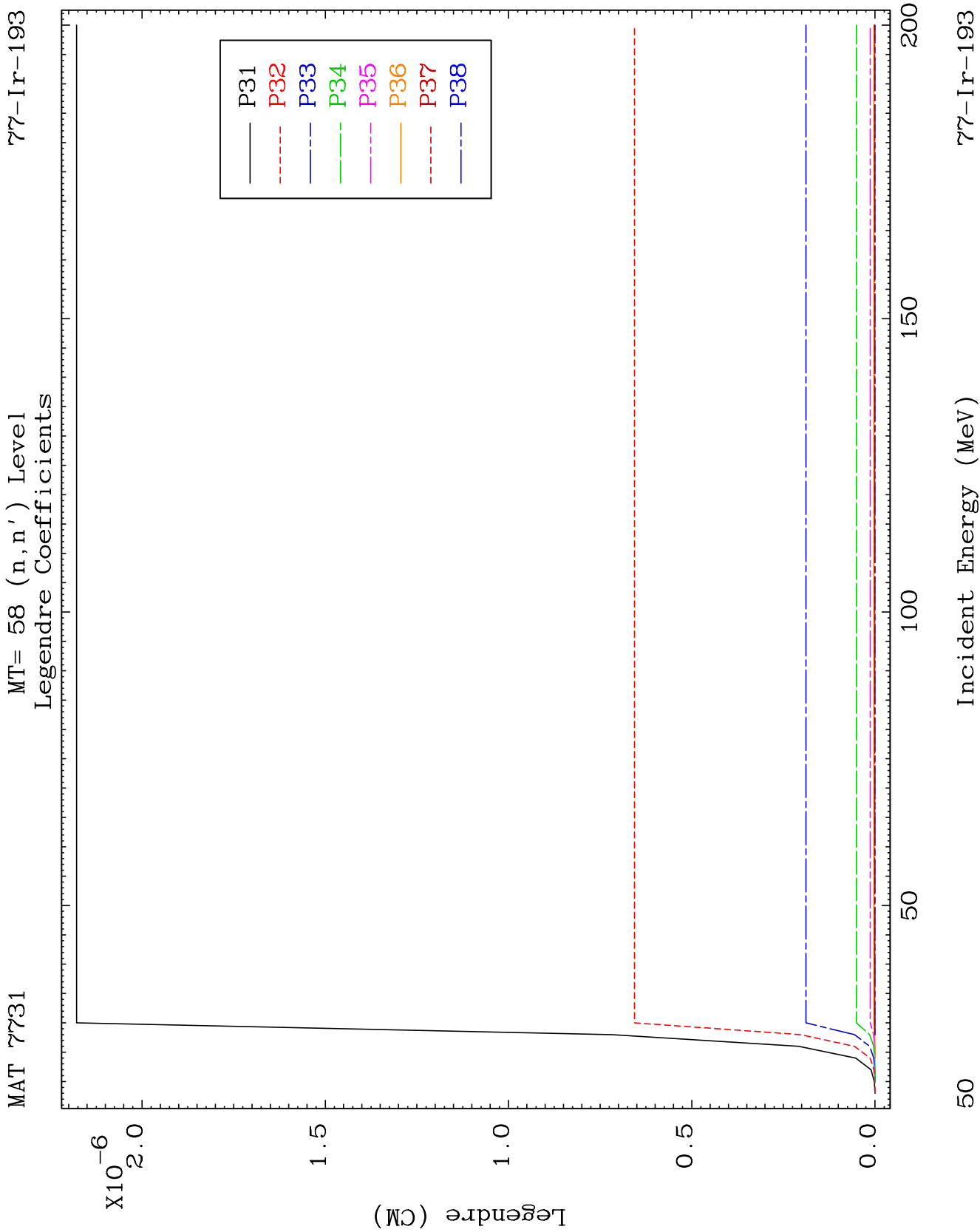


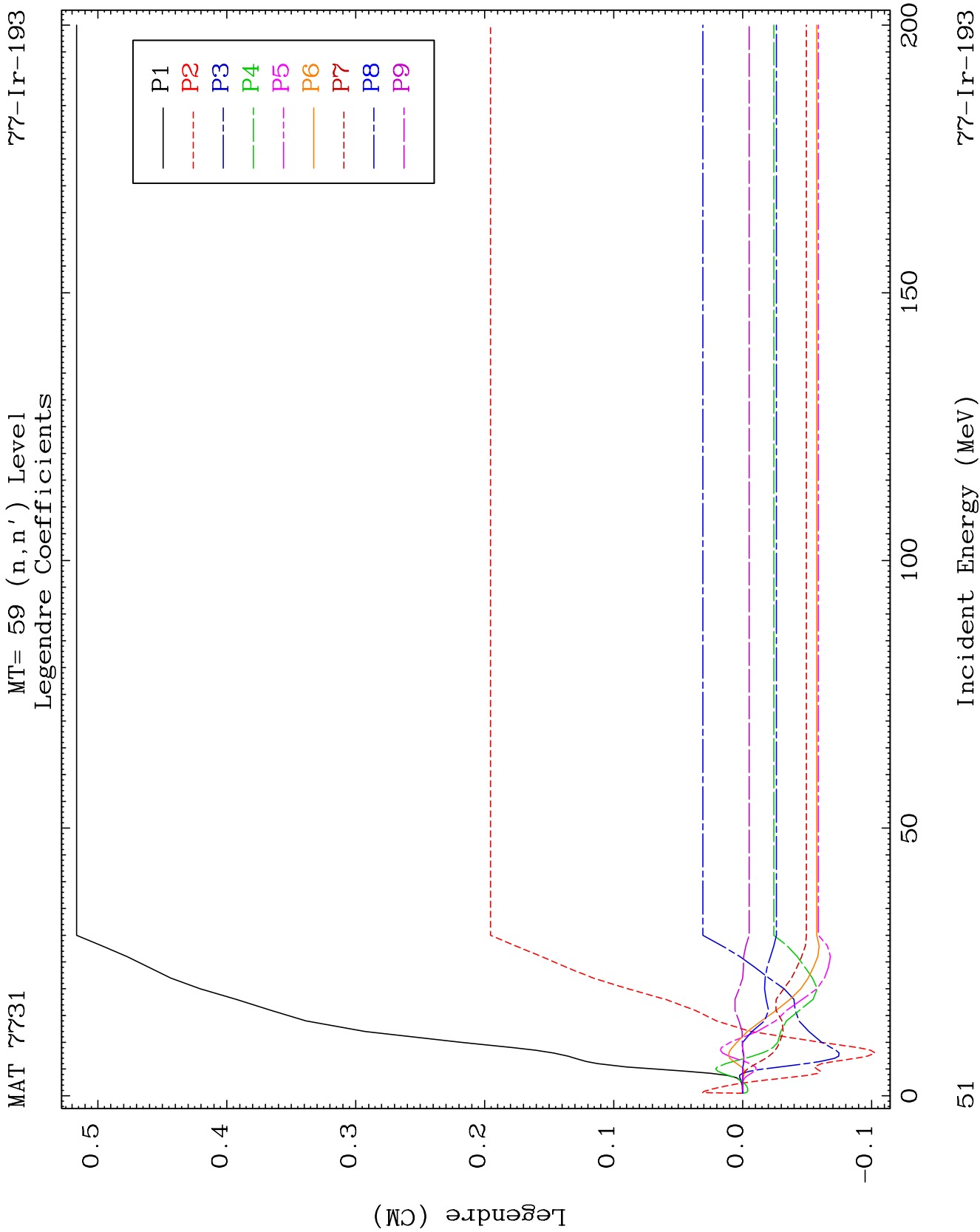
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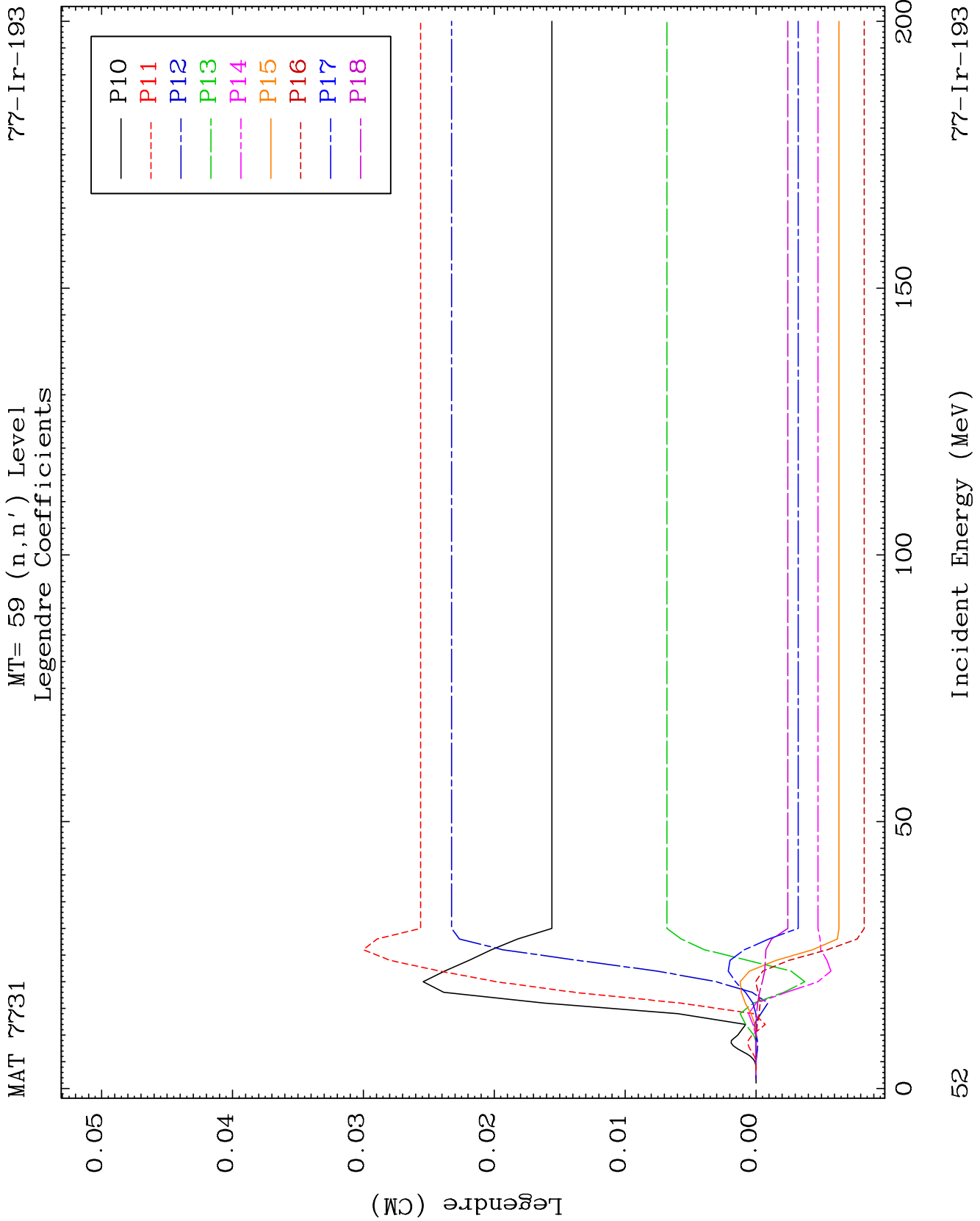
MT= 58 (n, n') Level
Legendre Coefficients

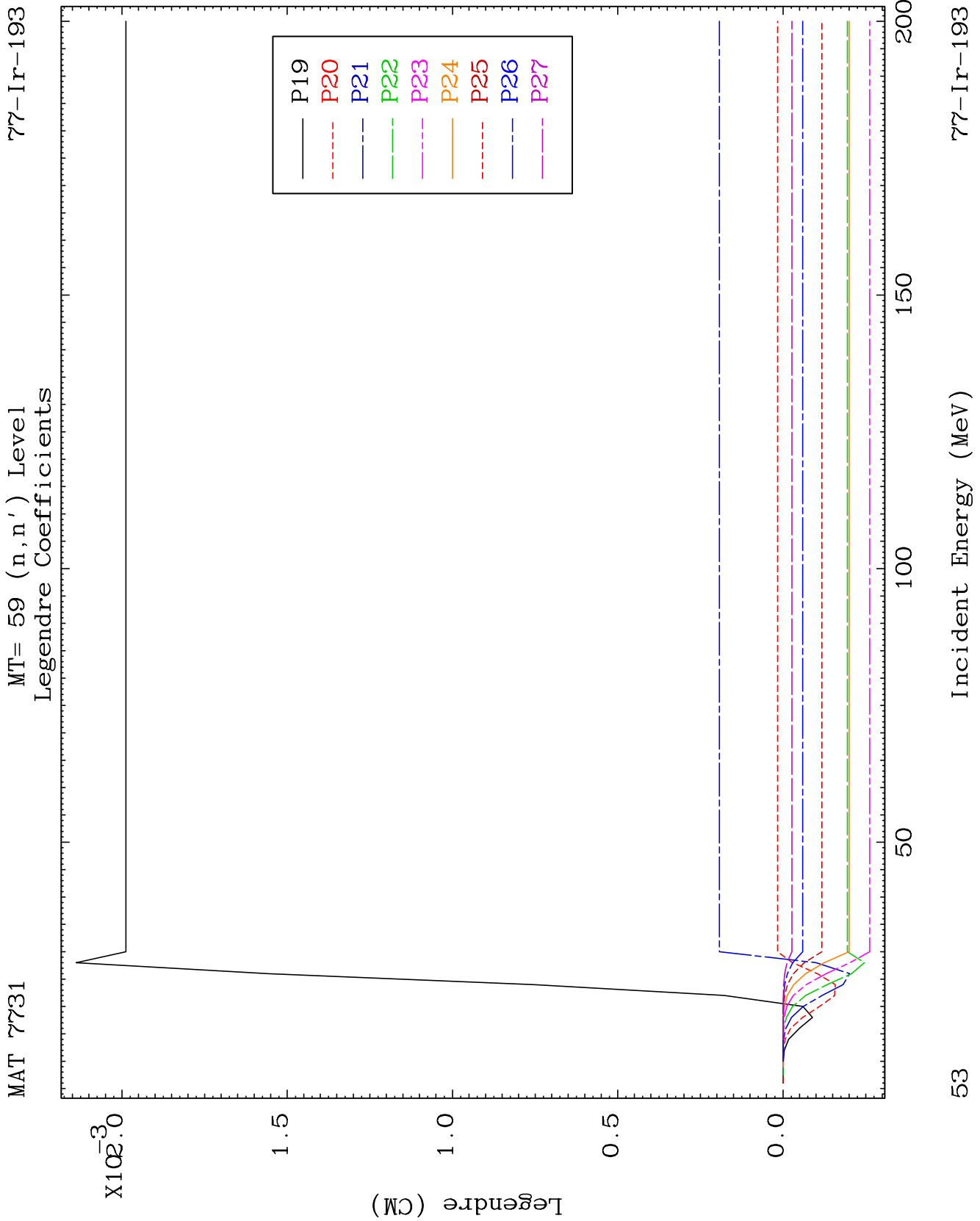
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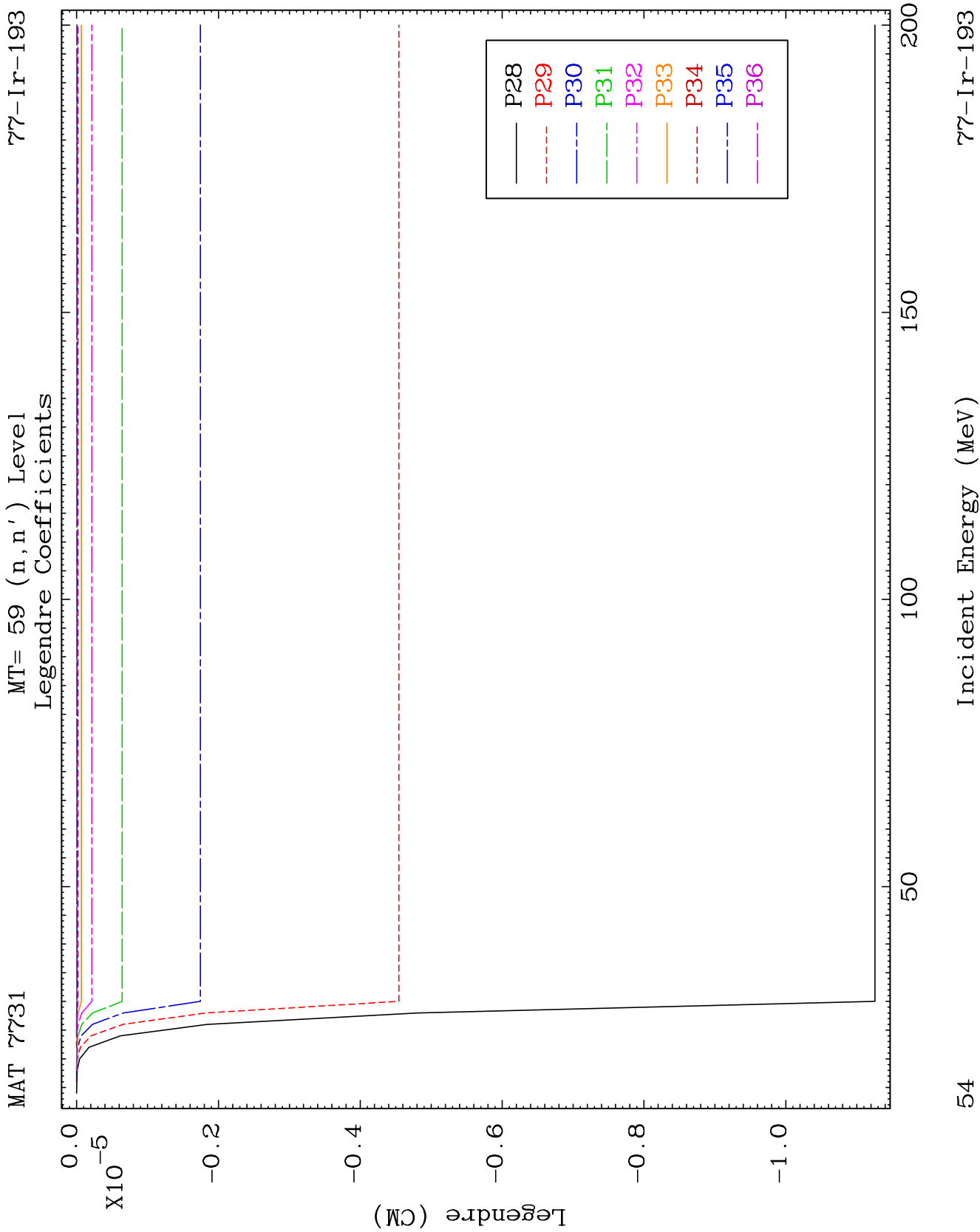


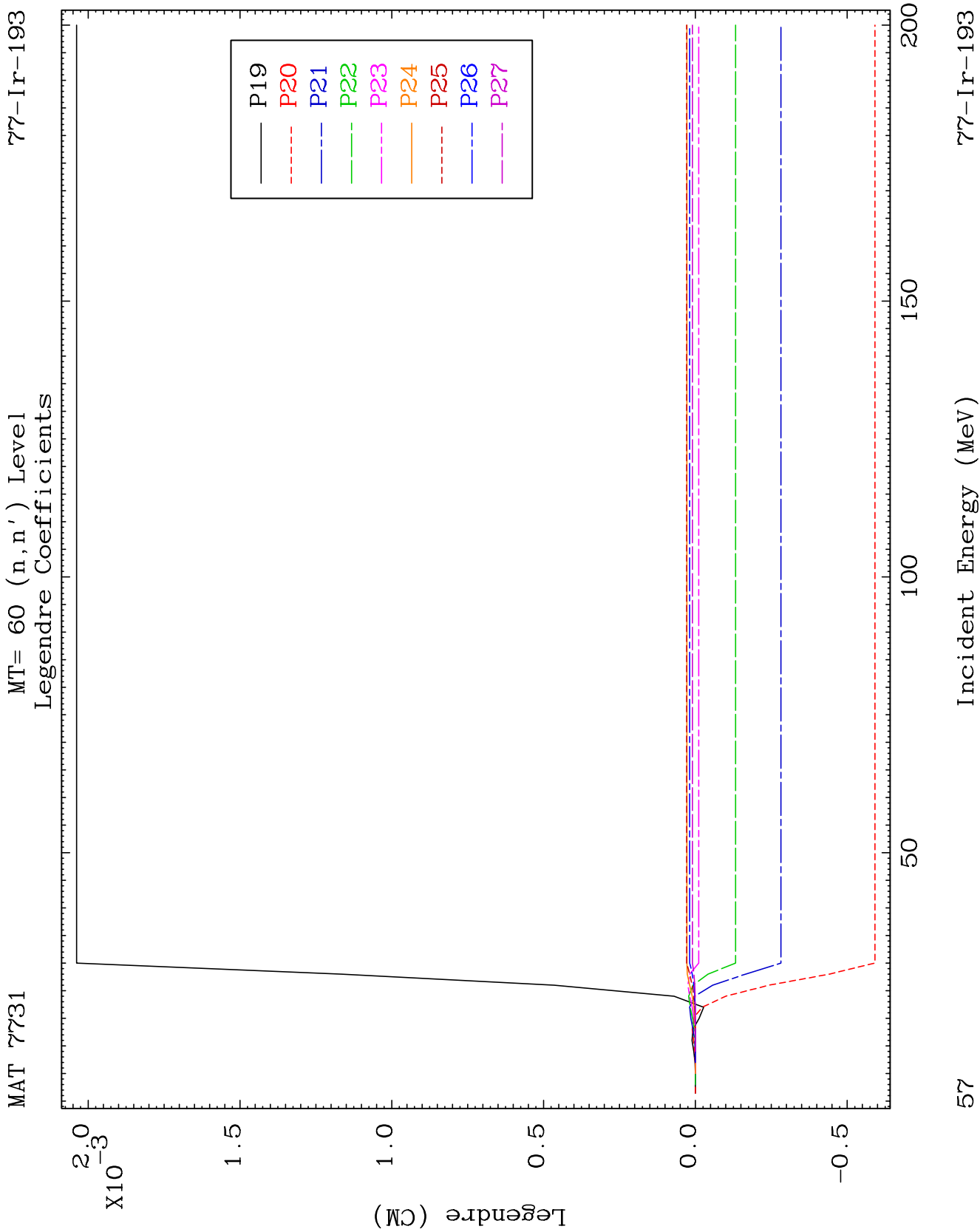








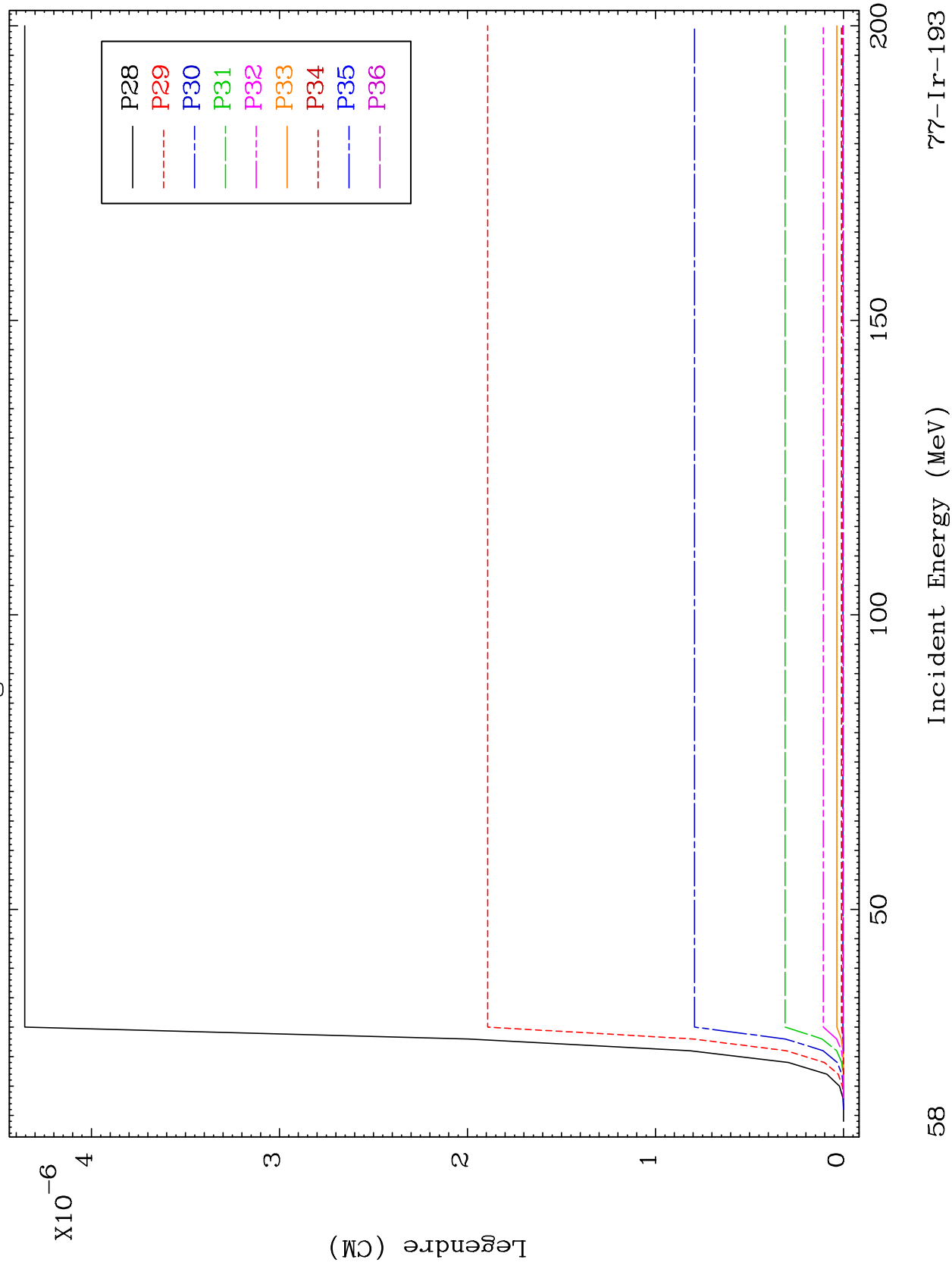


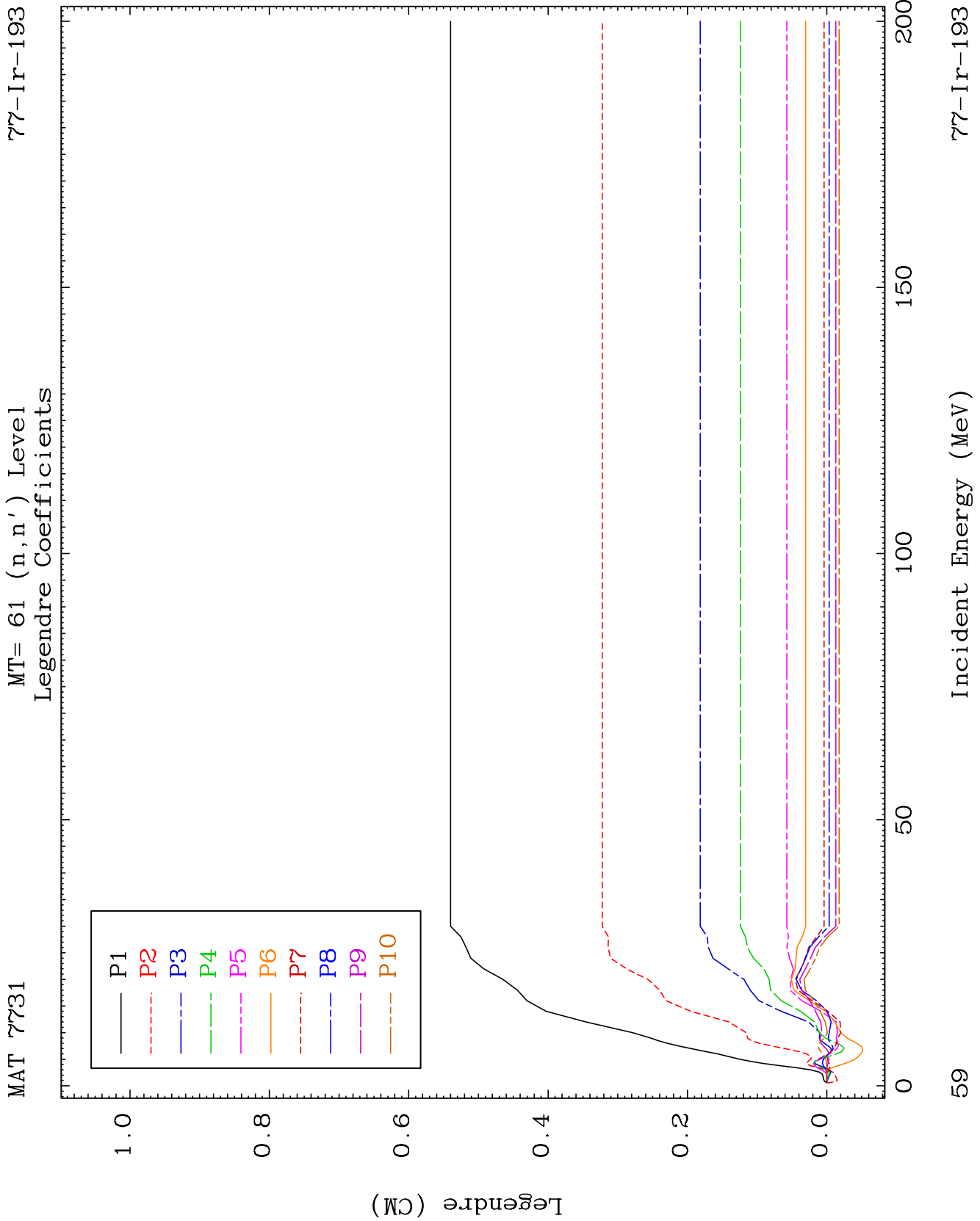


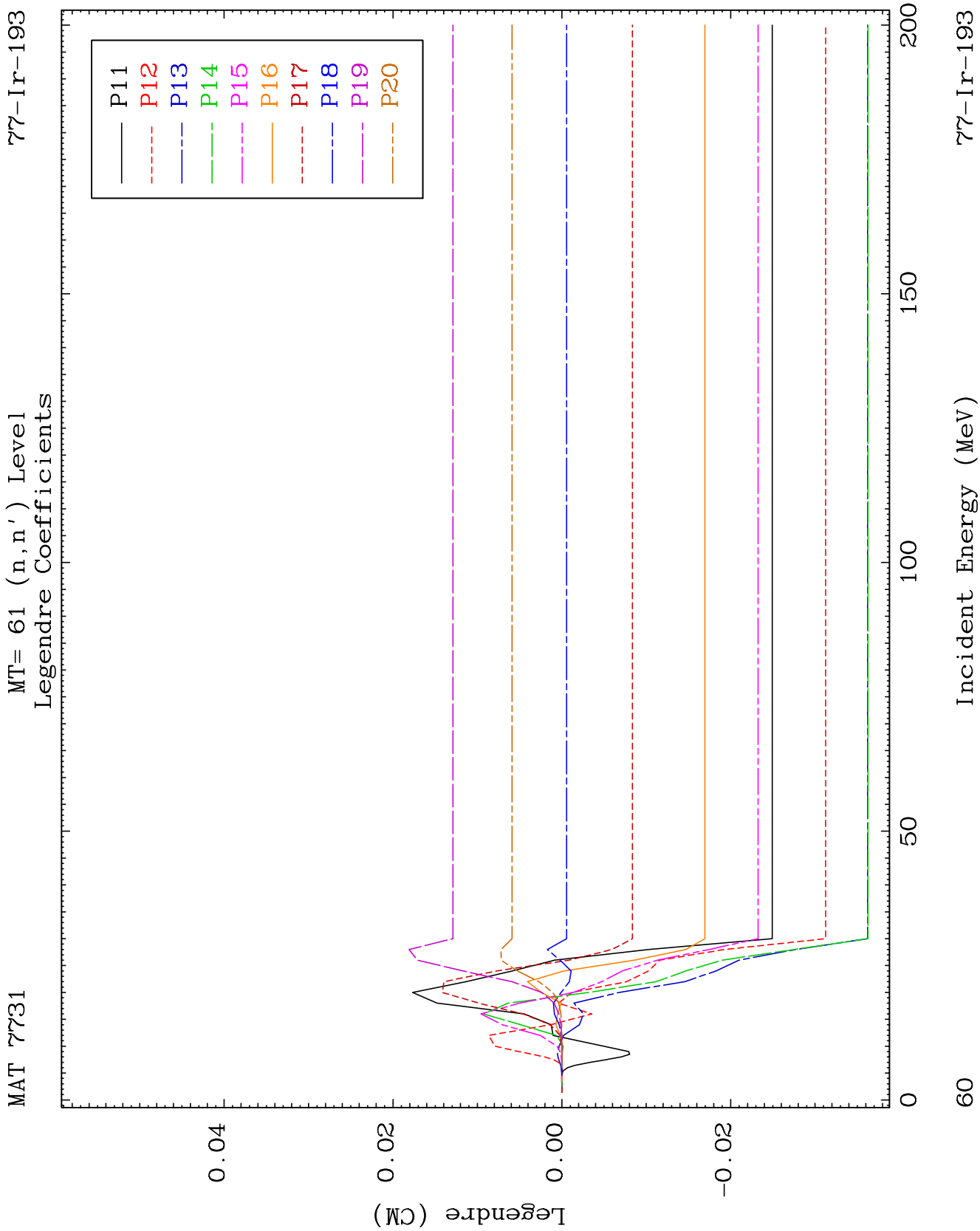
MAT 7731

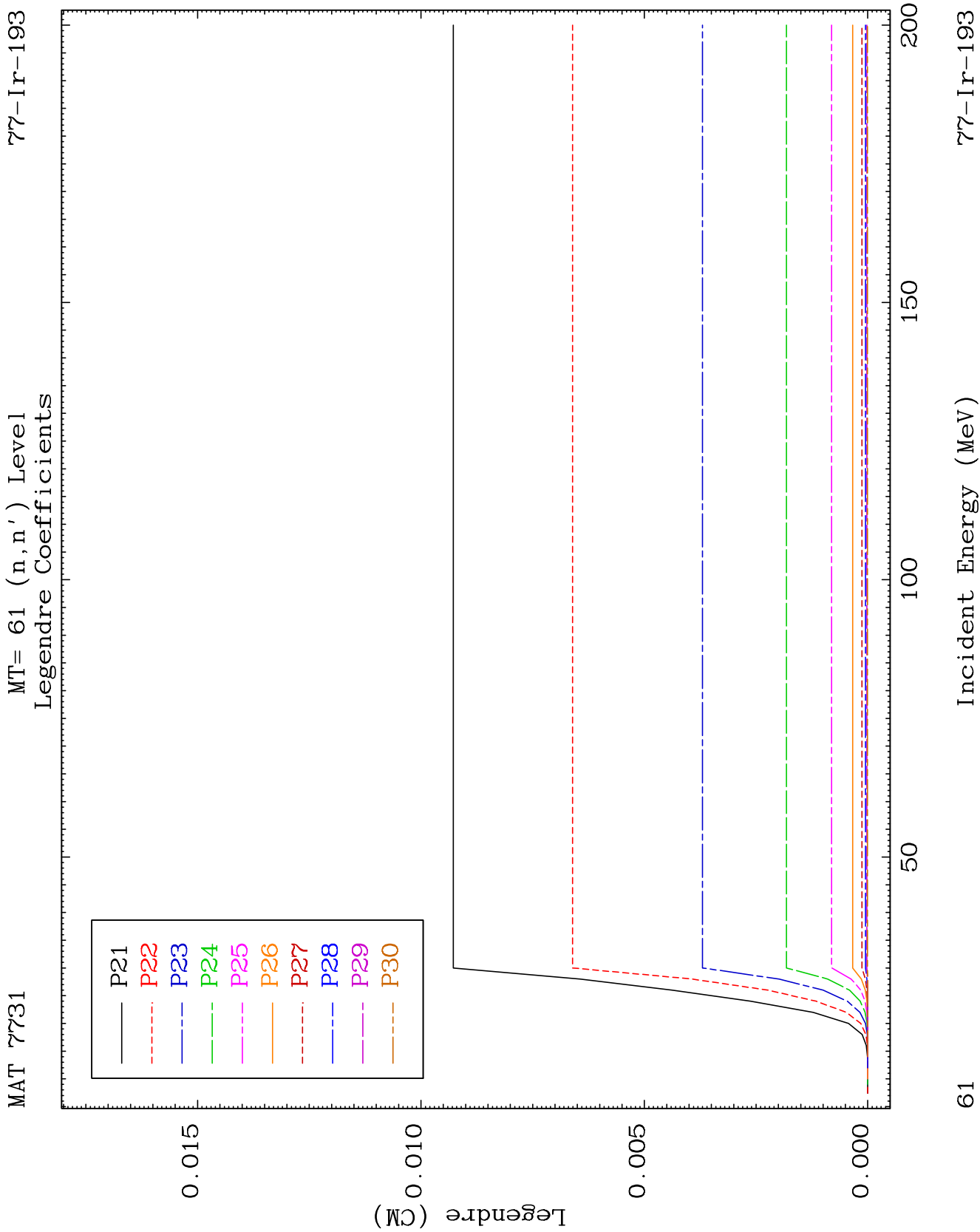
MT= 60 (n, n') Level
Legendre Coefficients

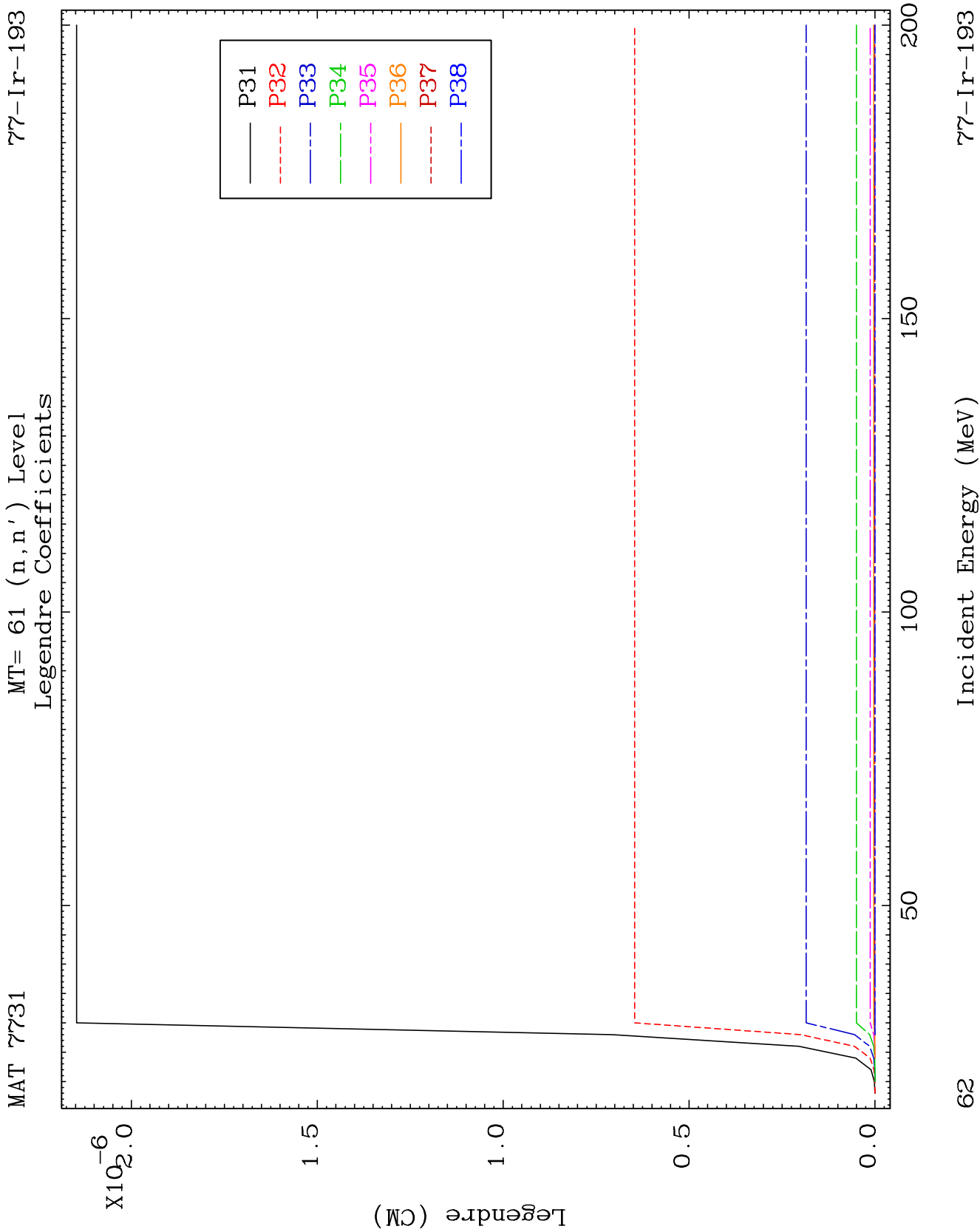
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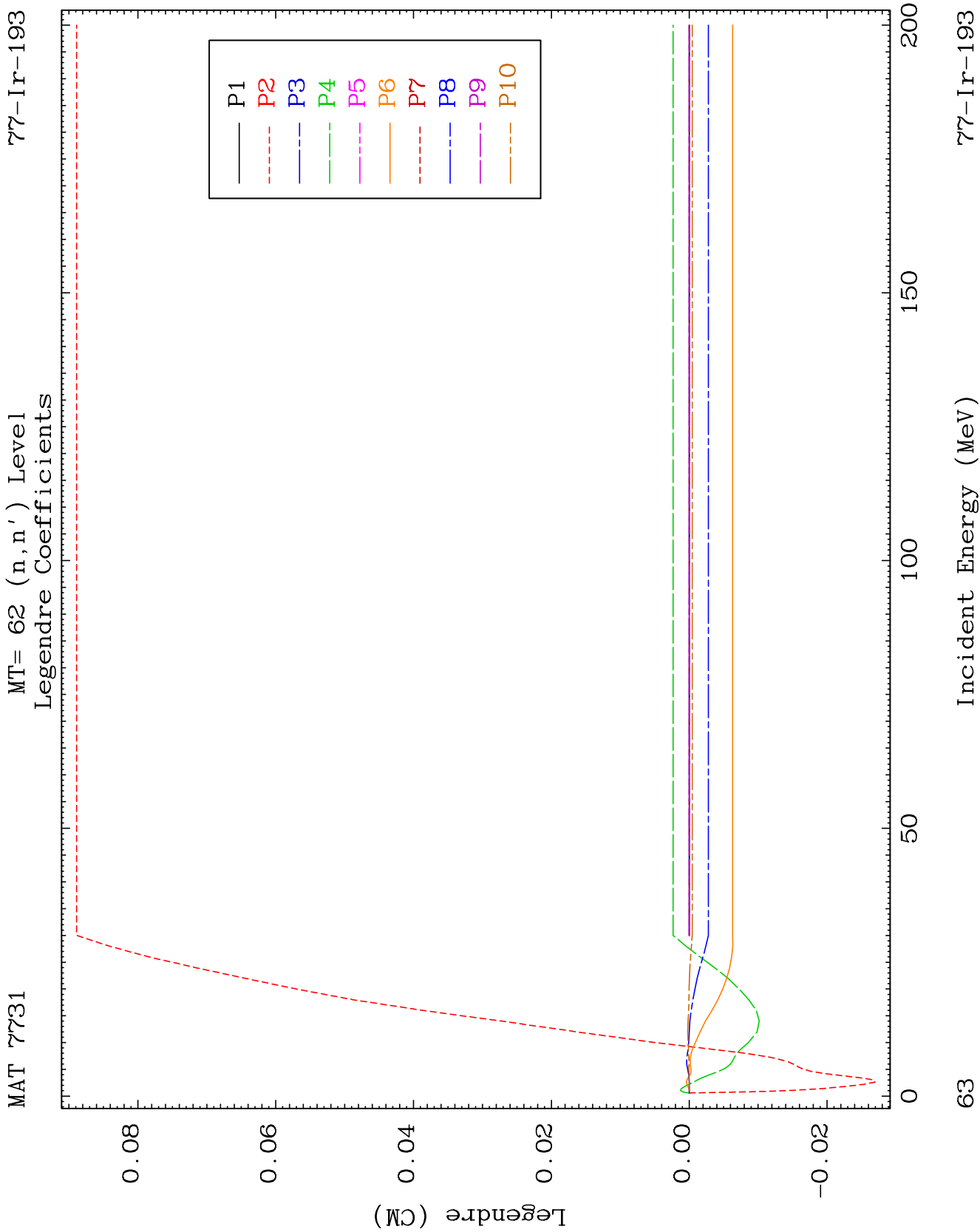


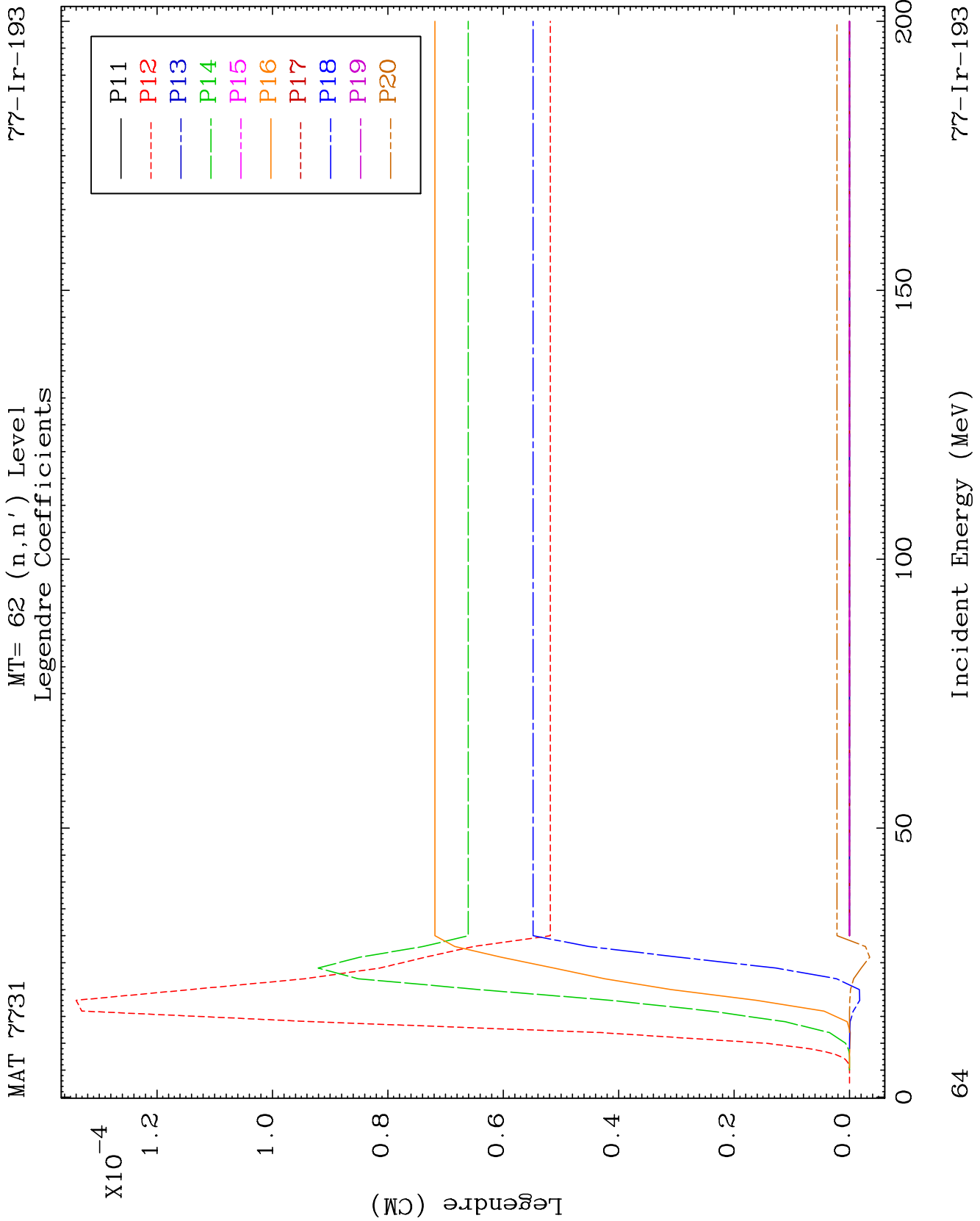


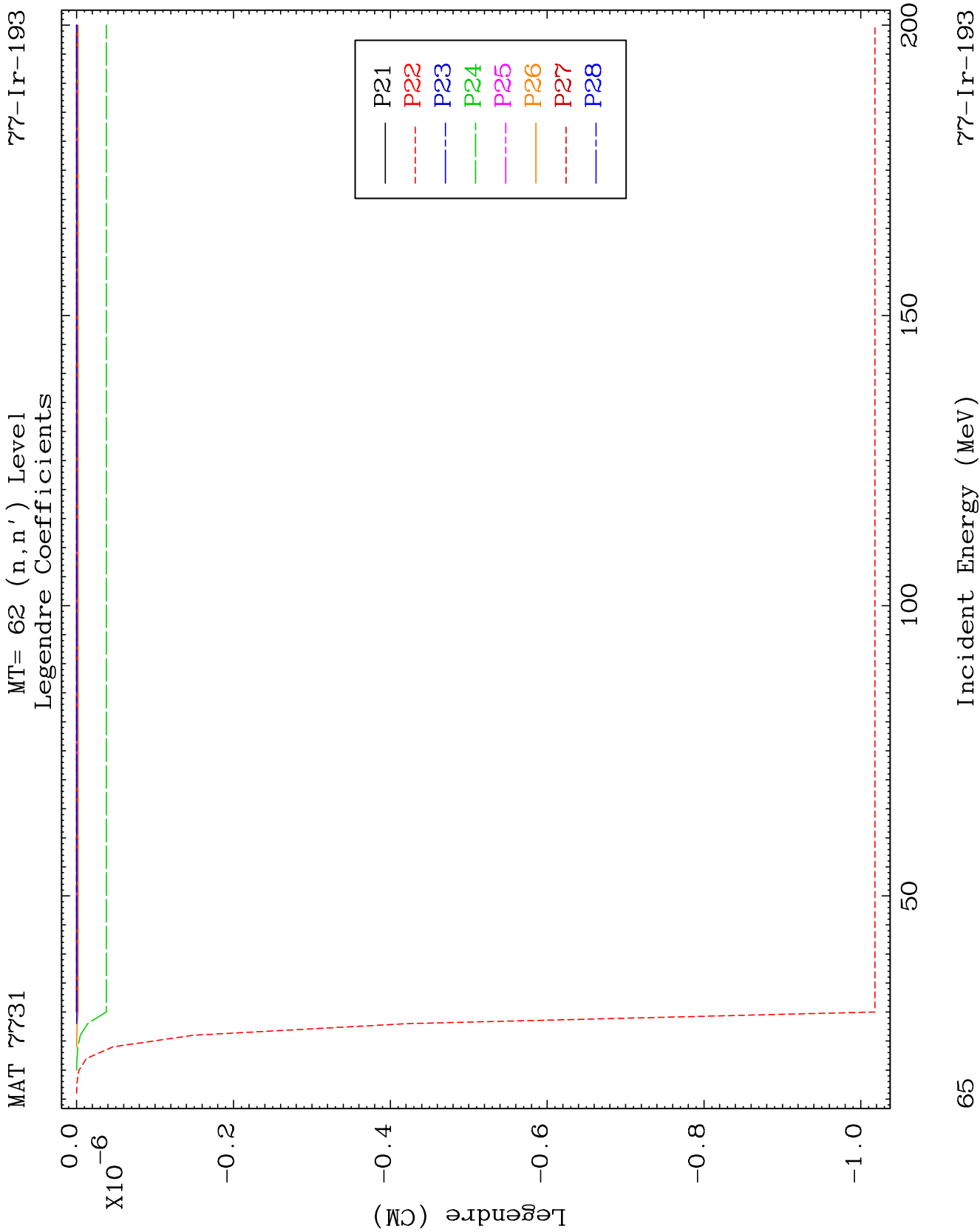


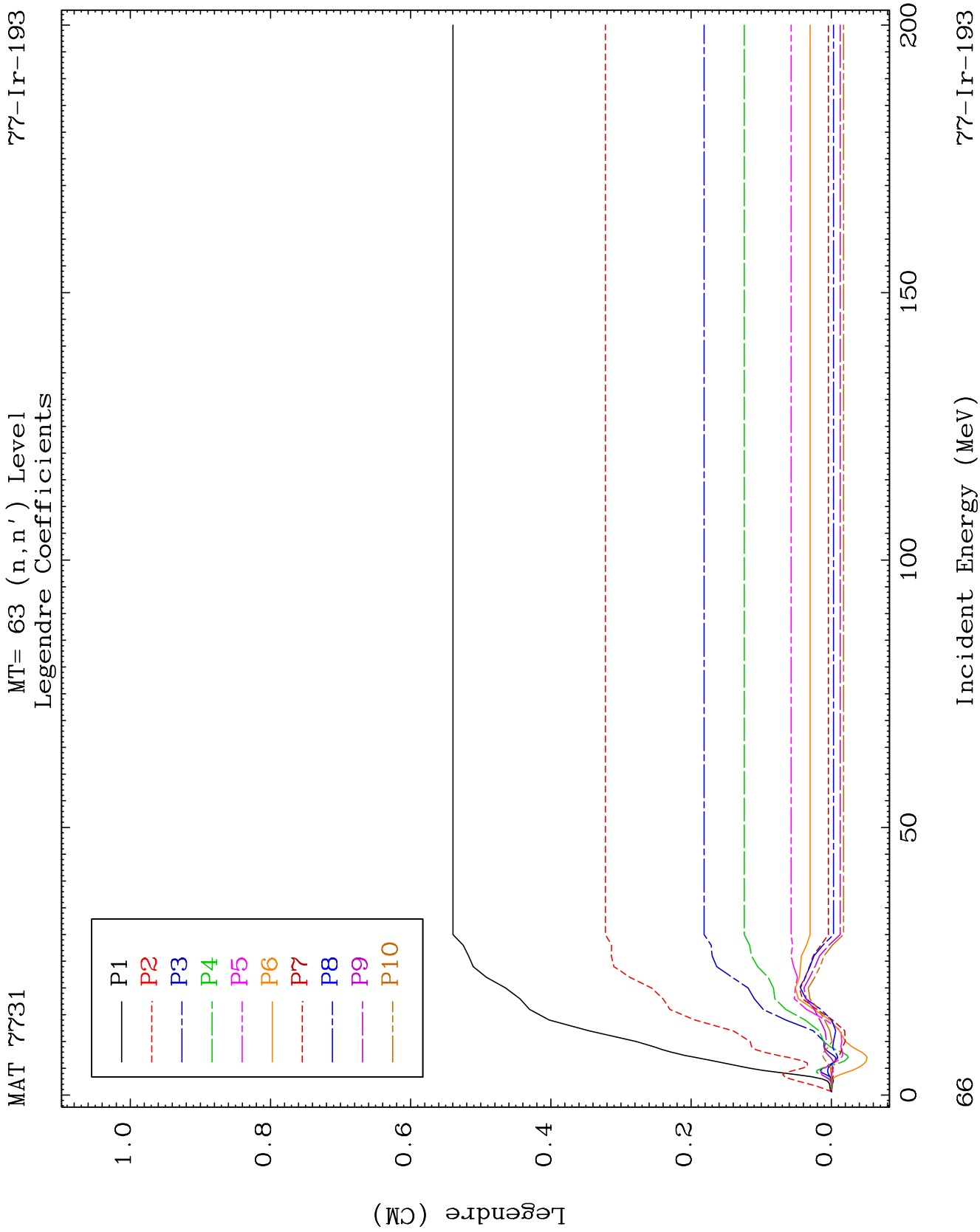


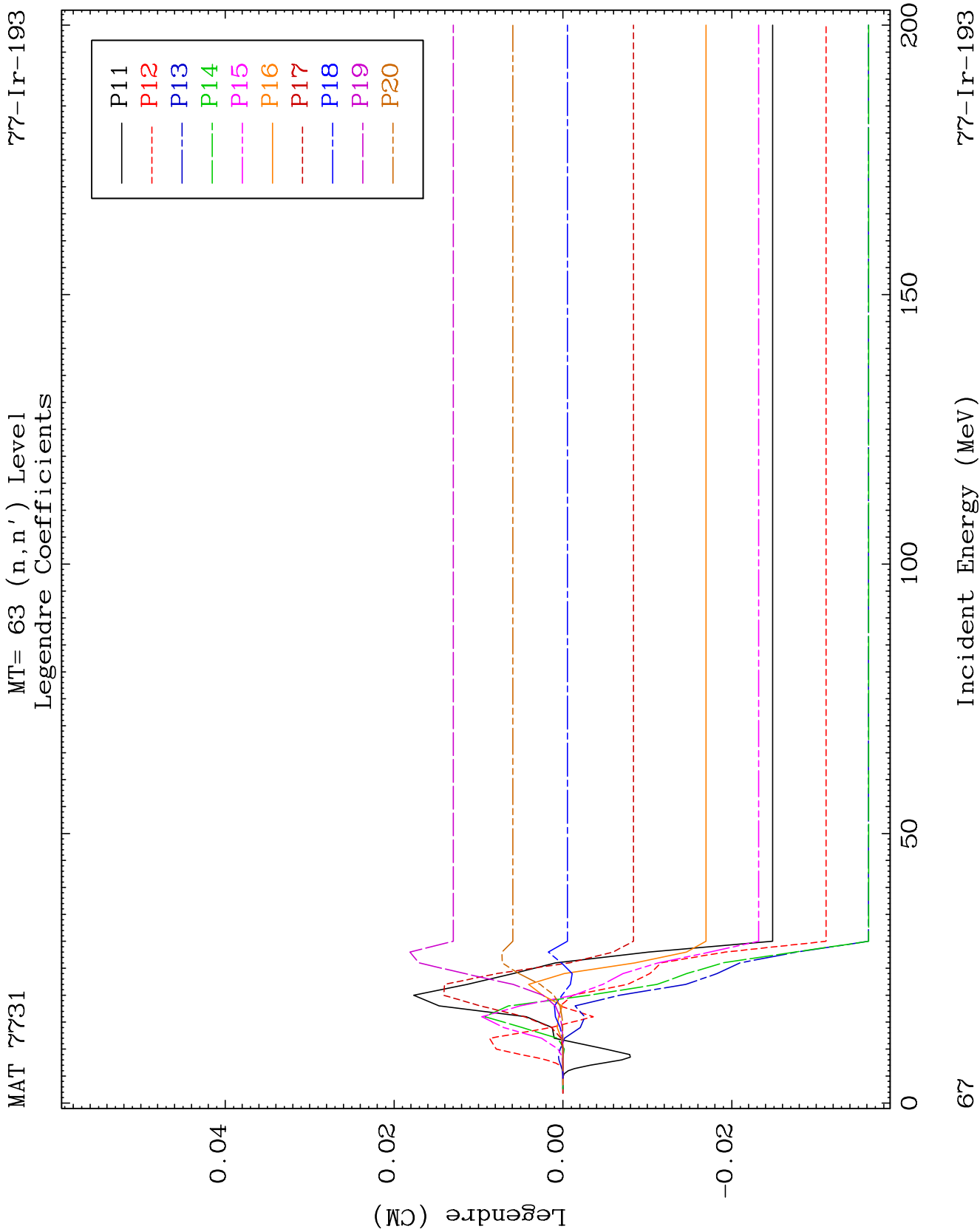


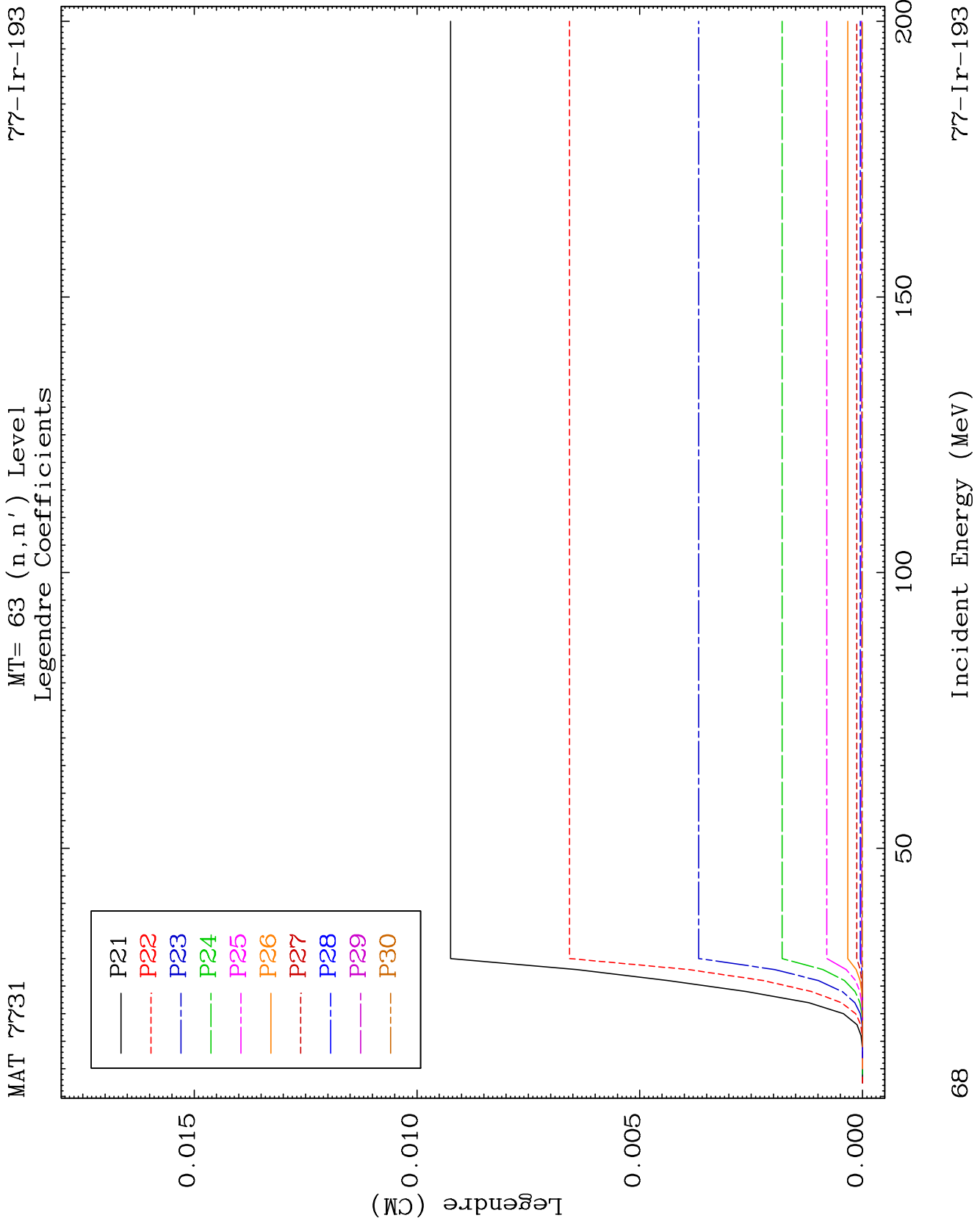








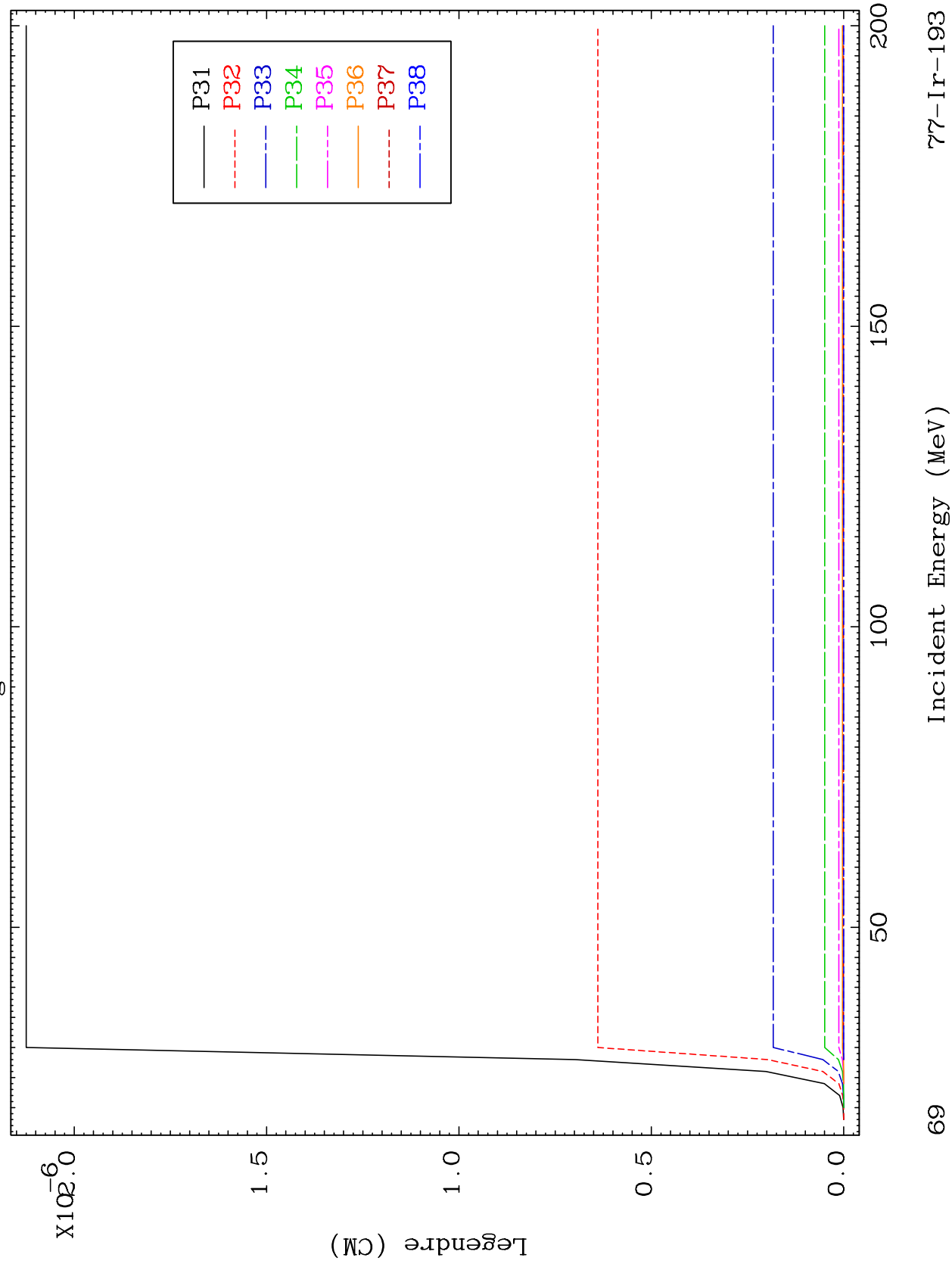


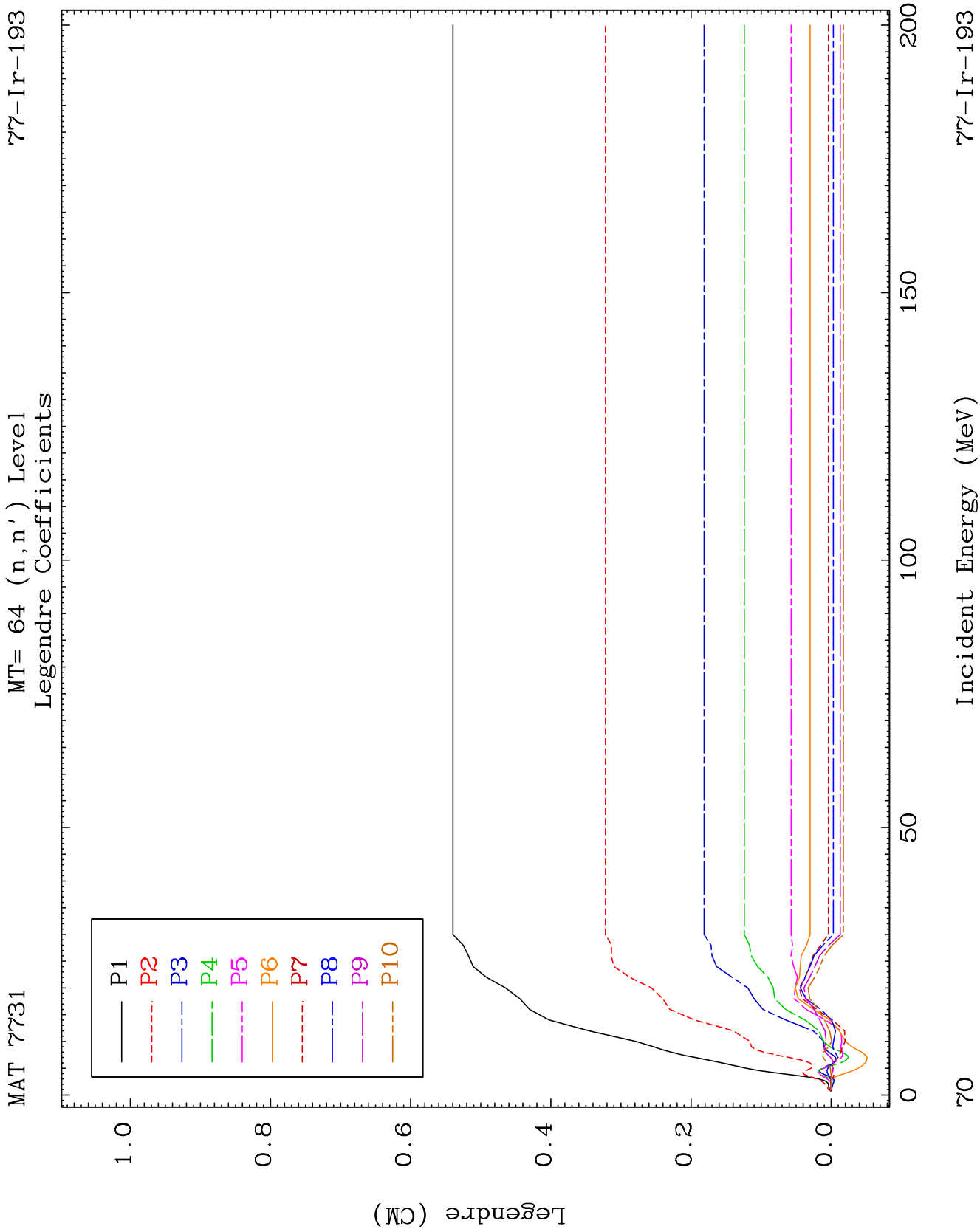


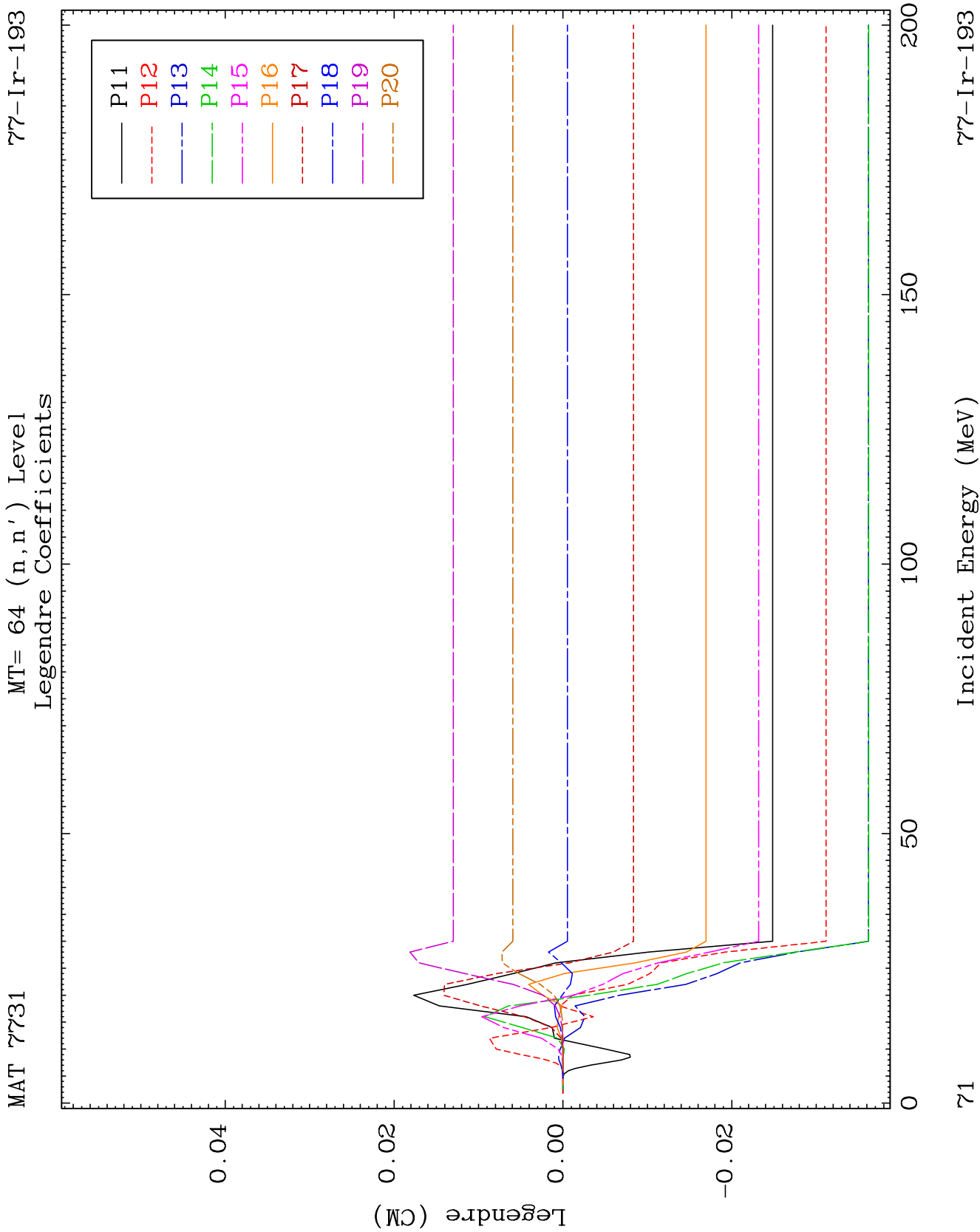
MAT 7731

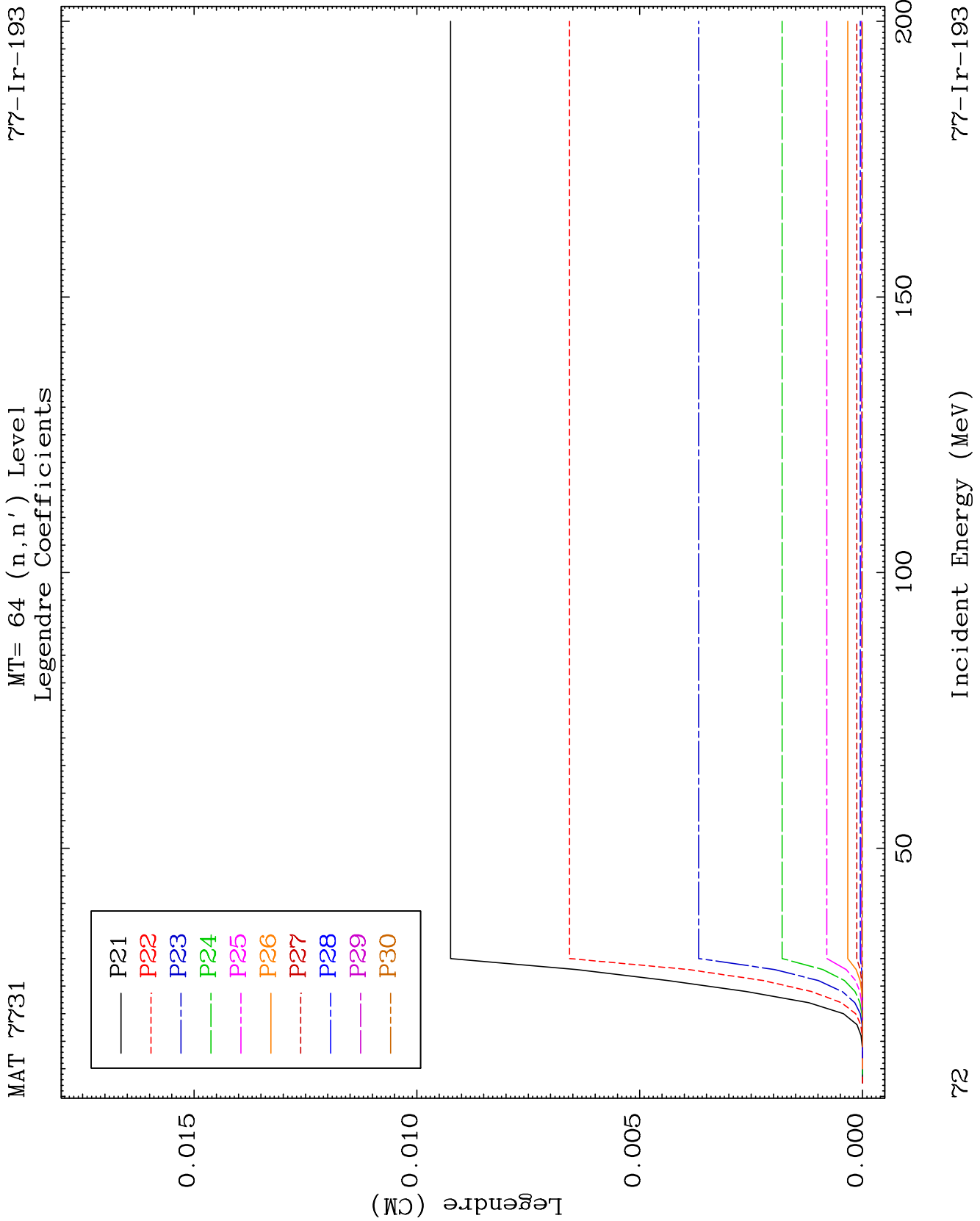
MT= 63 (n, n') Level
Legendre Coefficients

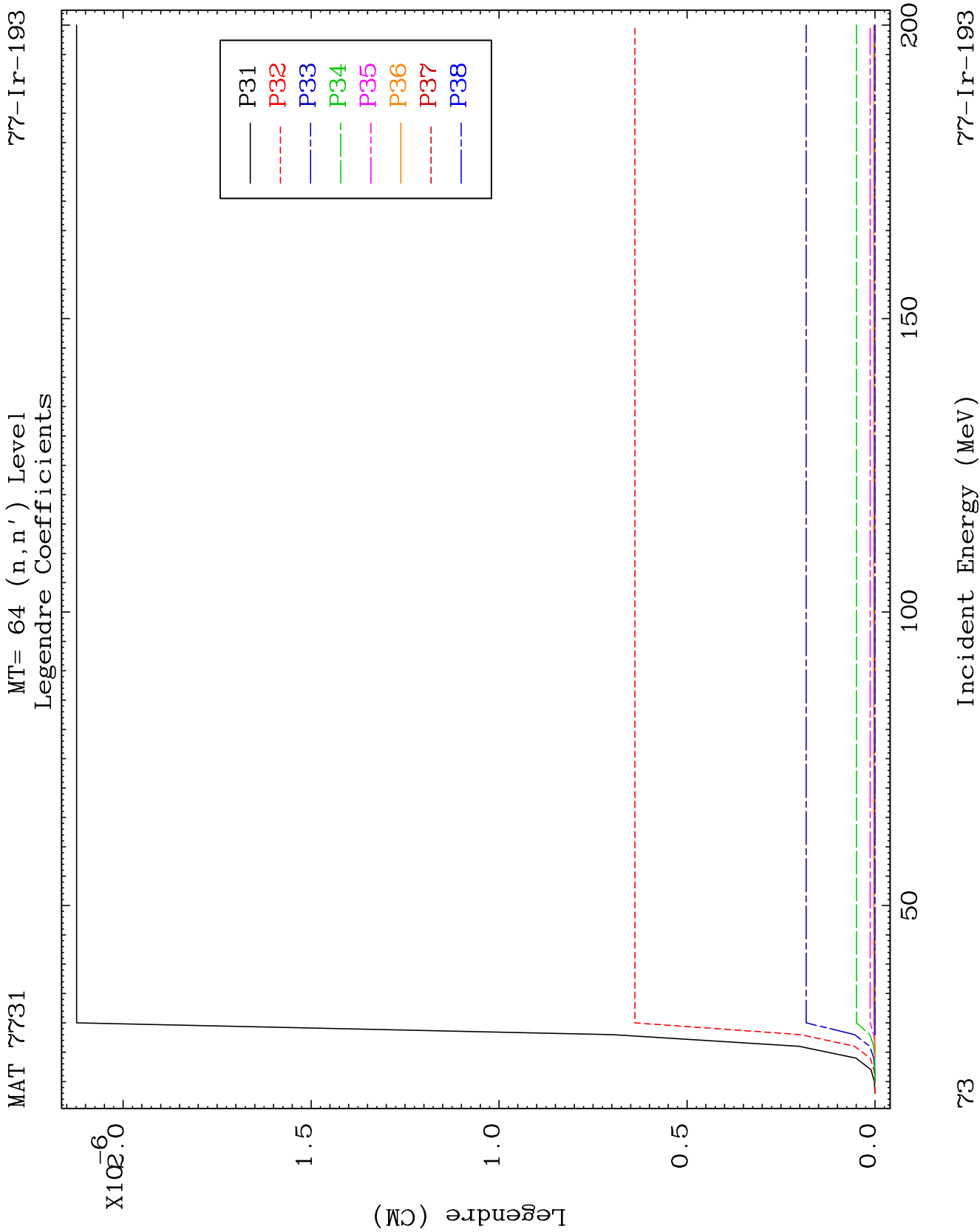
77-Ir-193







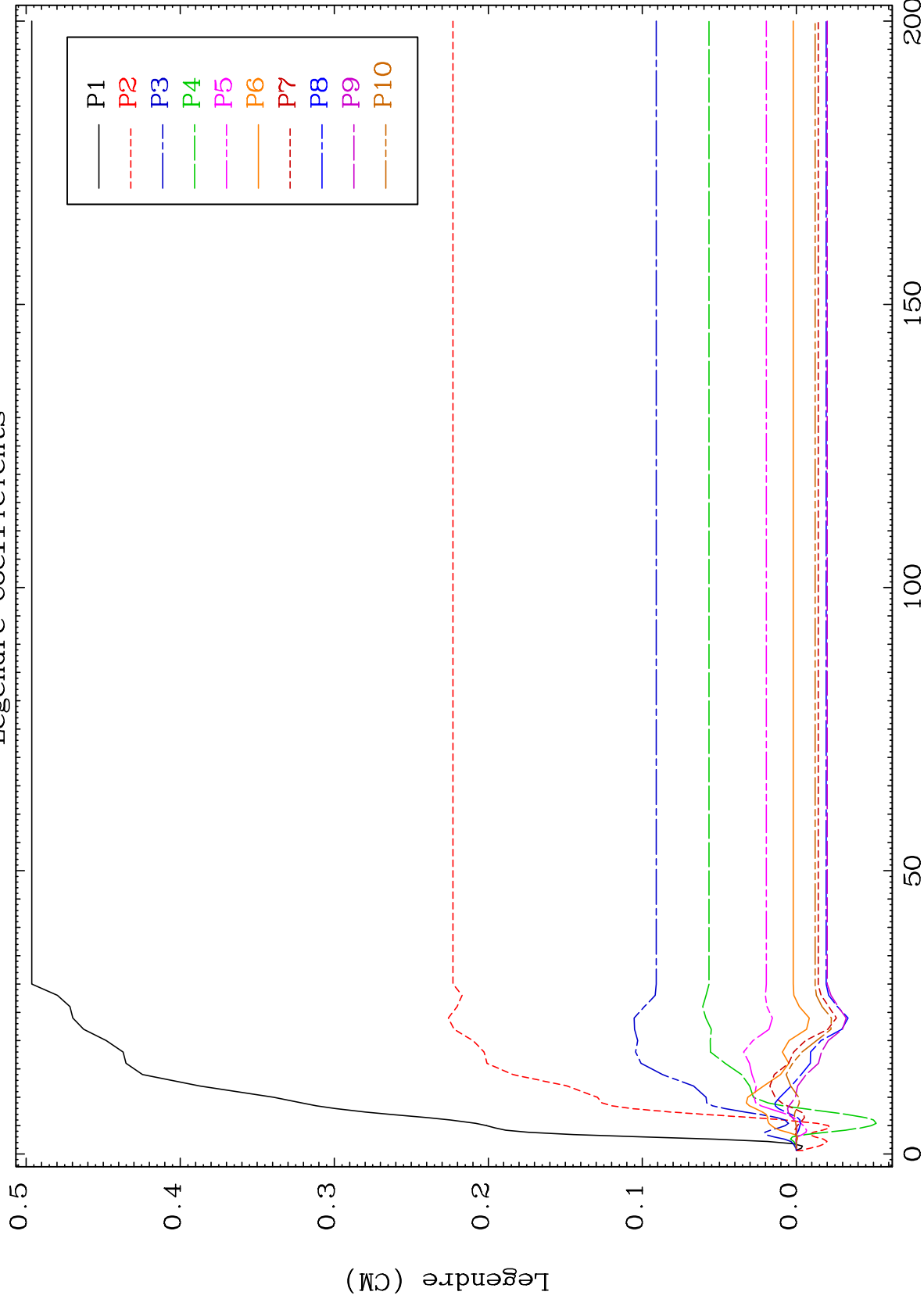




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MT= 65 (n, n') Level
Legendre Coefficients

77-Ir-193



42

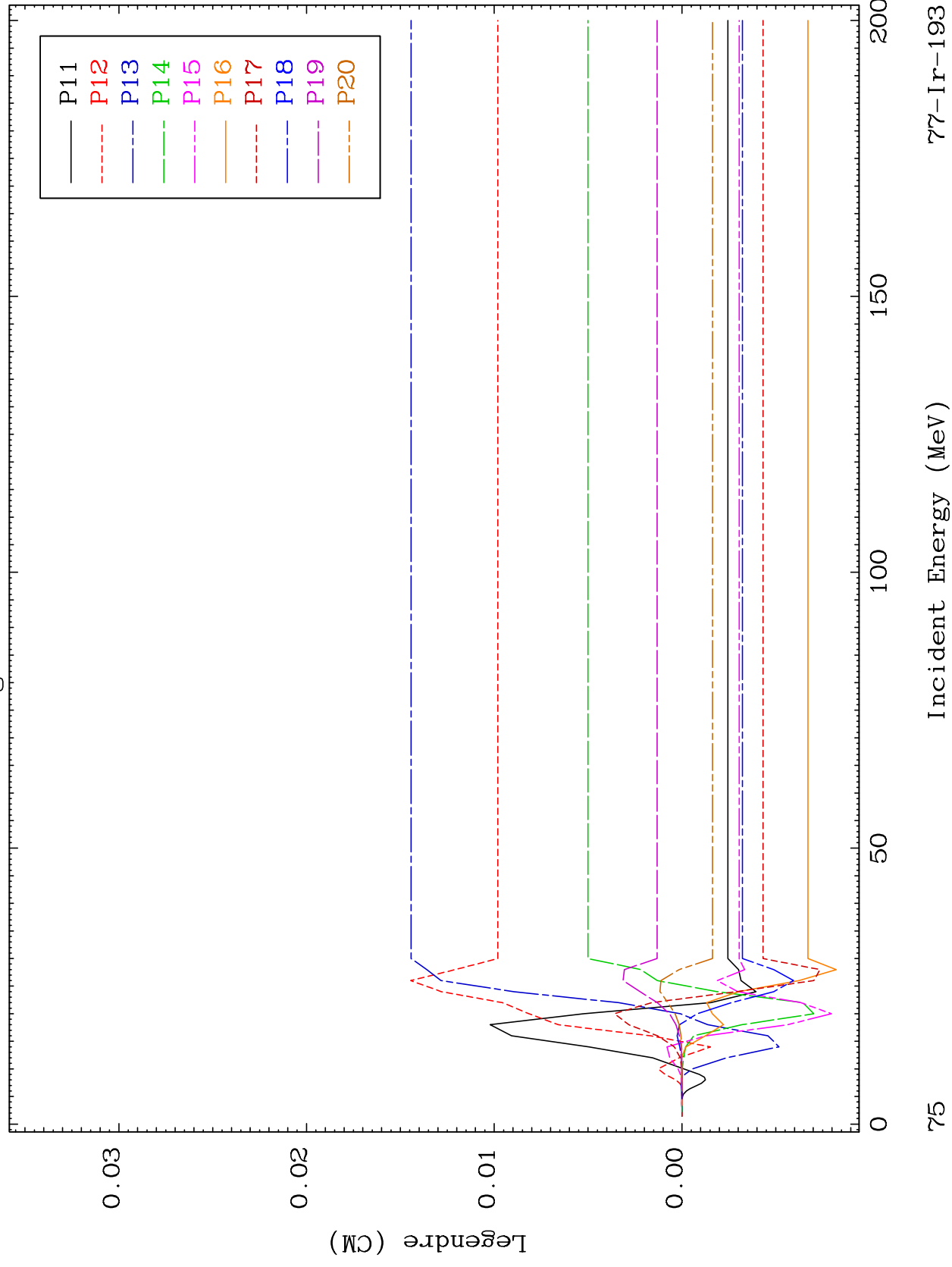
Incident Energy (MeV)

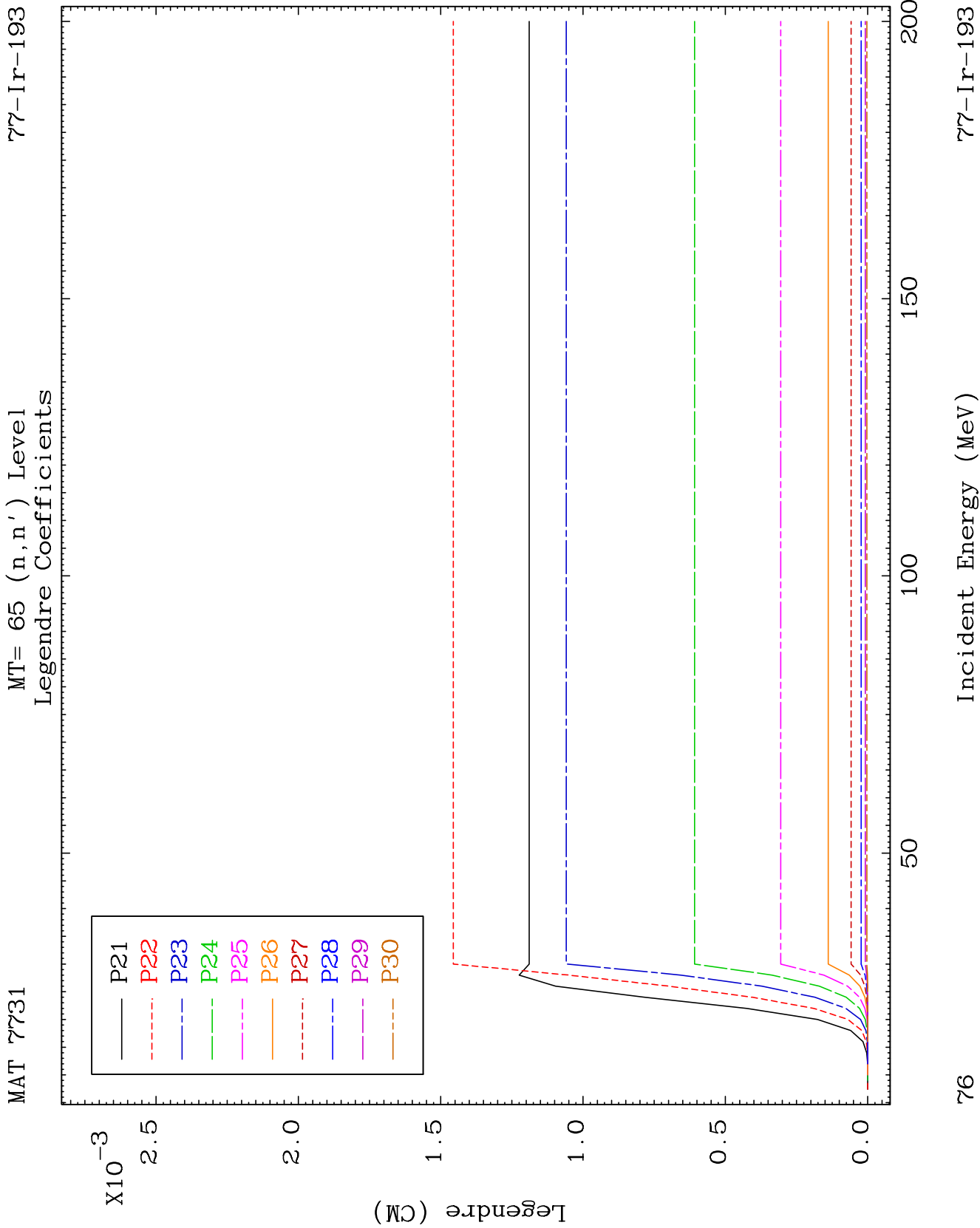
77-1r-193

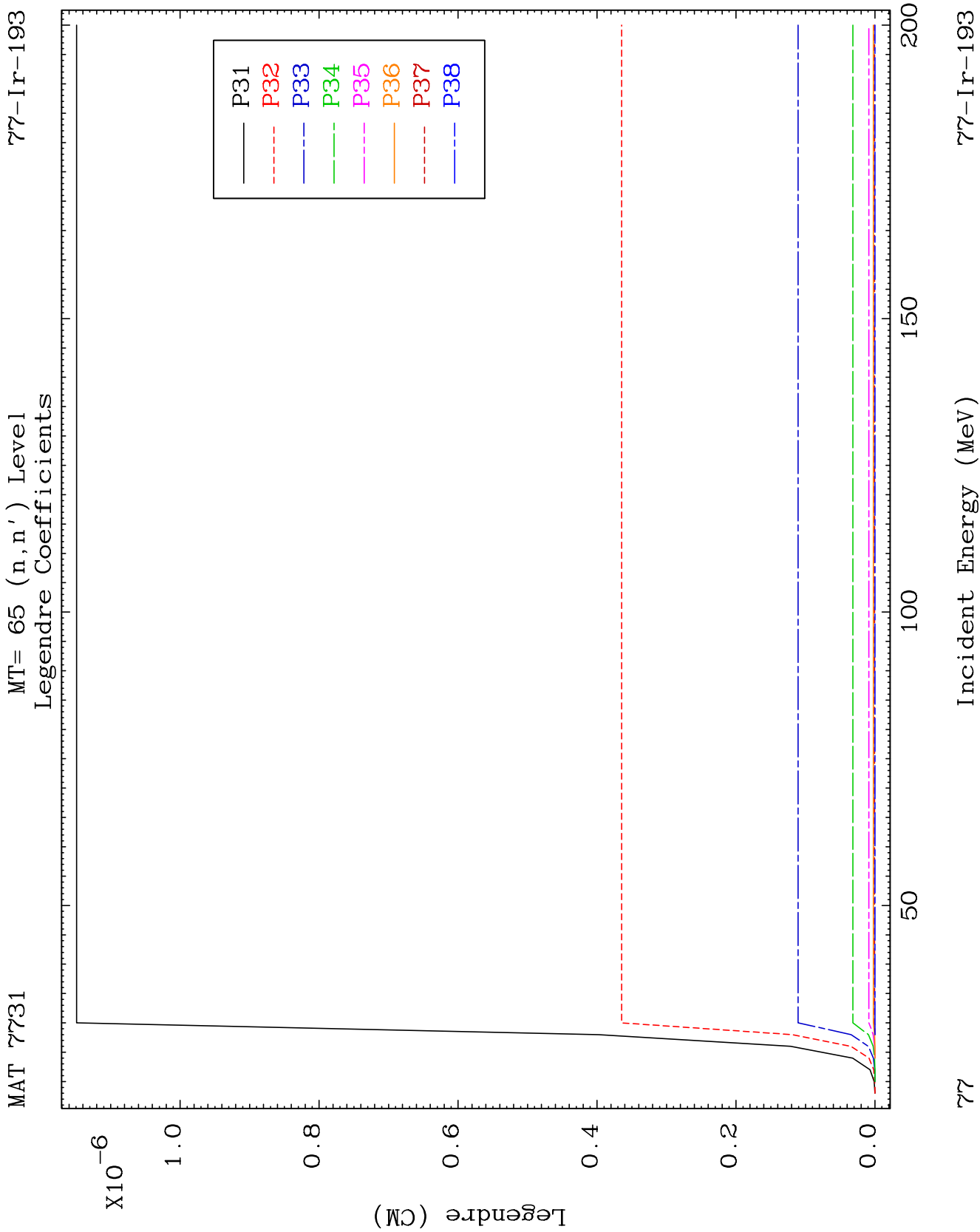
MAT 7731

MT= 65 (n, n') Level
Legendre Coefficients

77-Ir-193



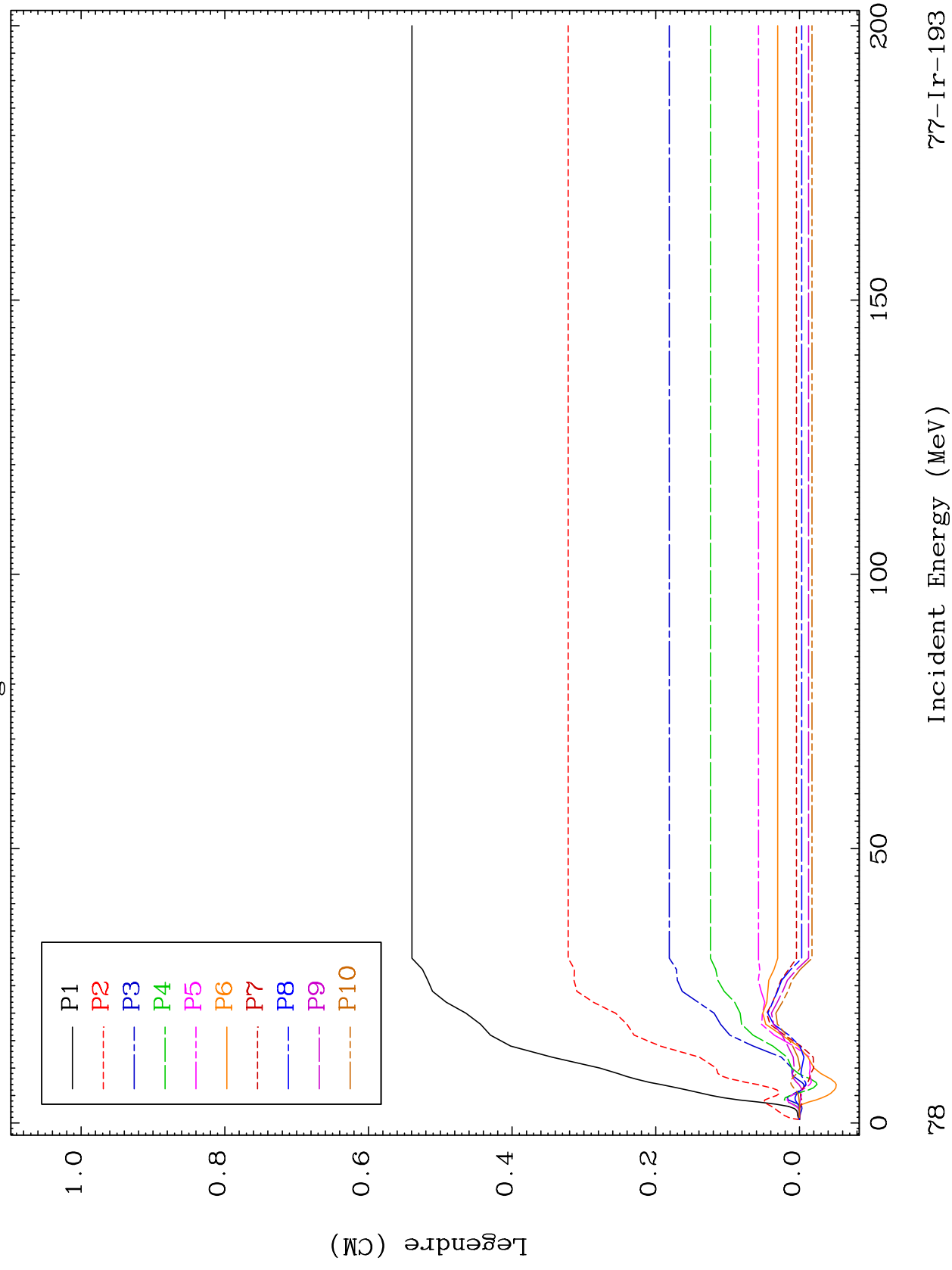


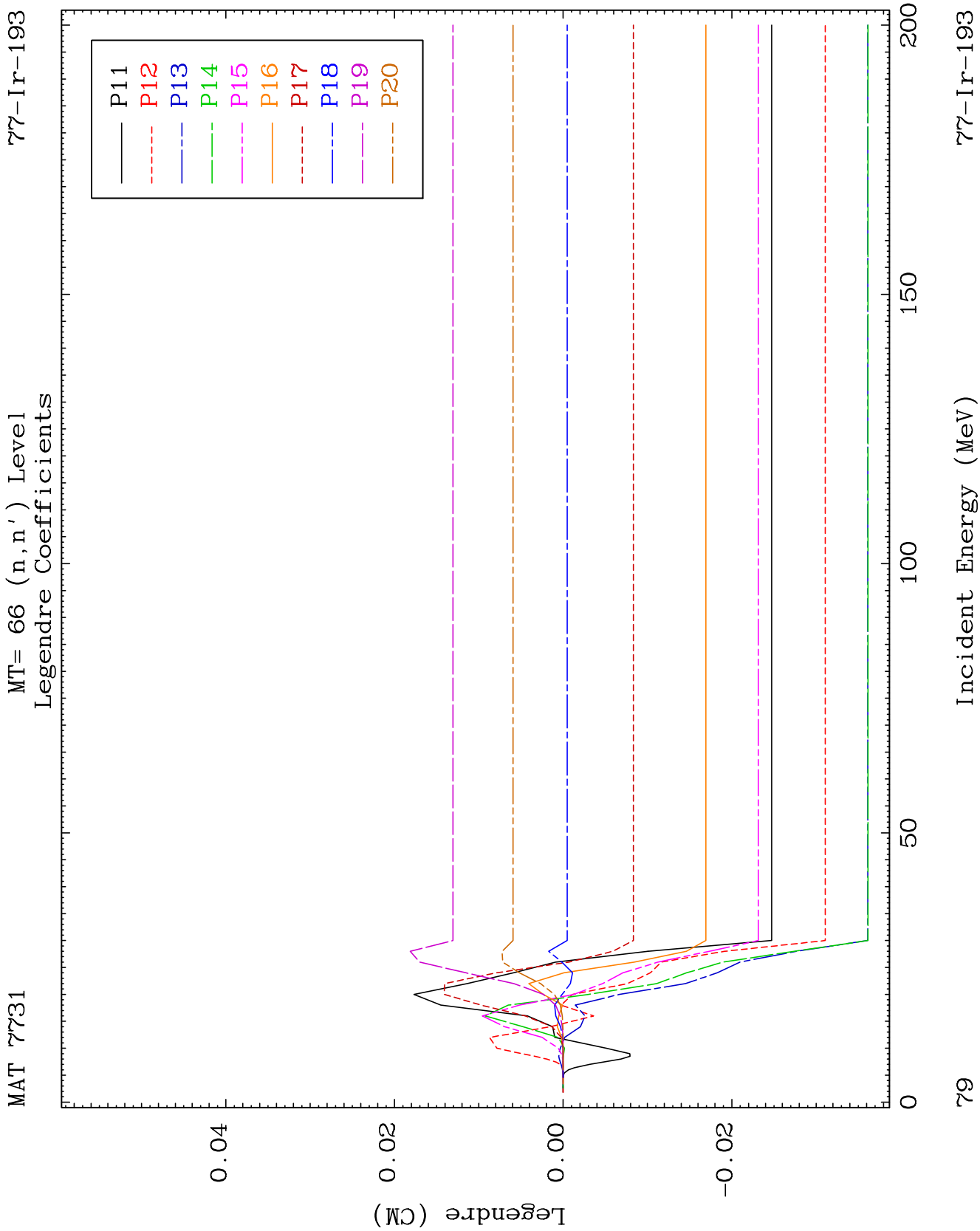


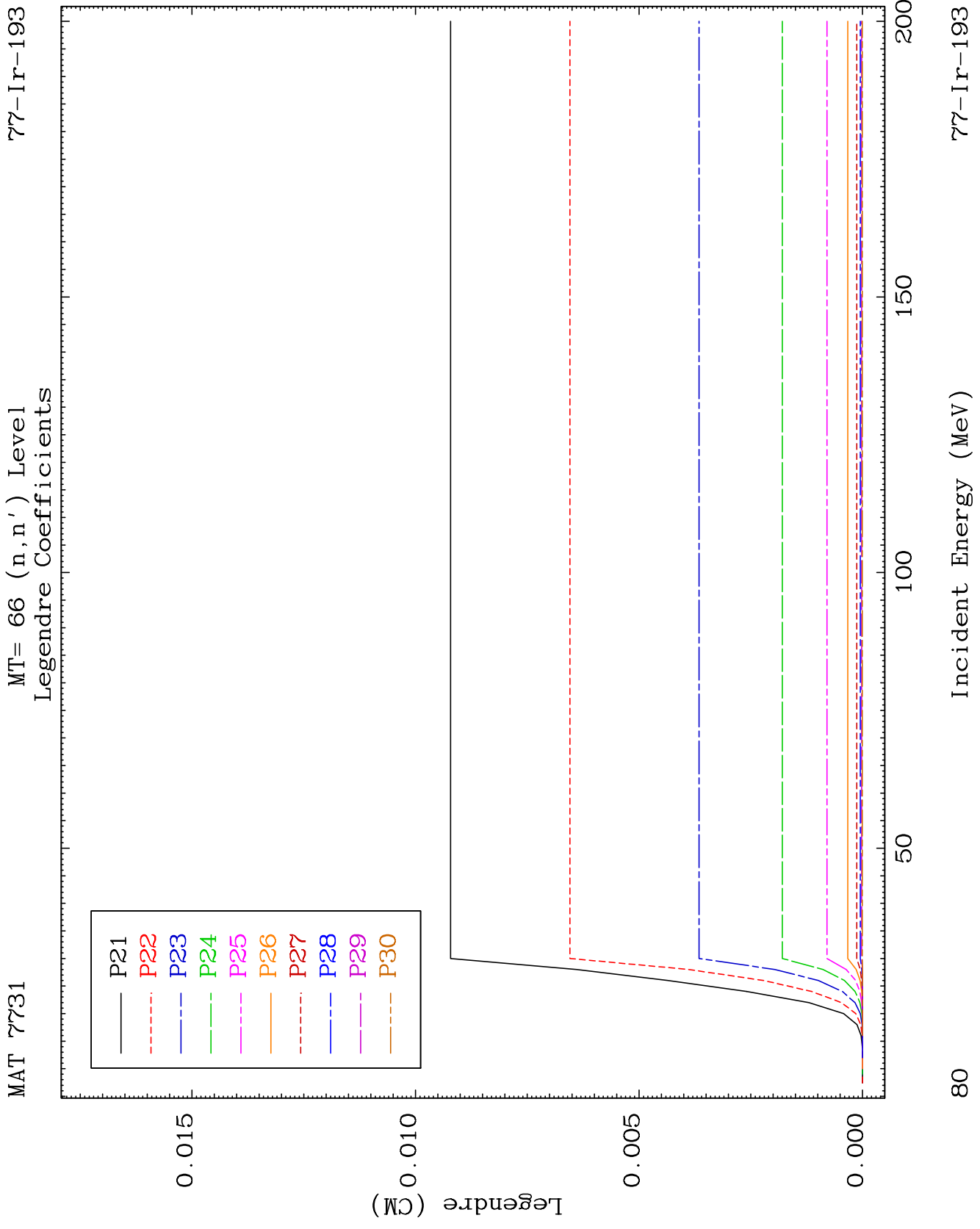
MAT 7731

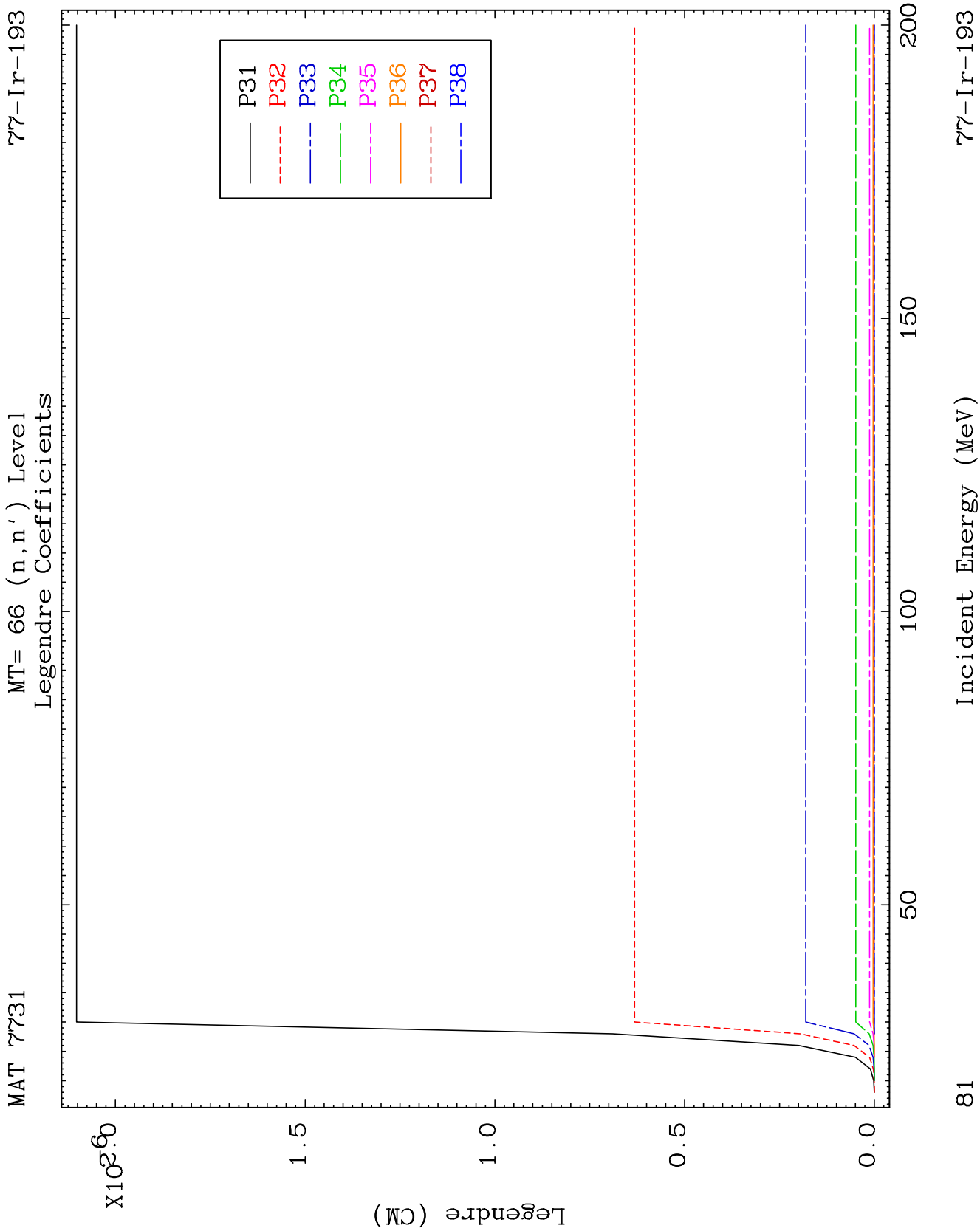
MT= 66 (n, n') Level
Legendre Coefficients

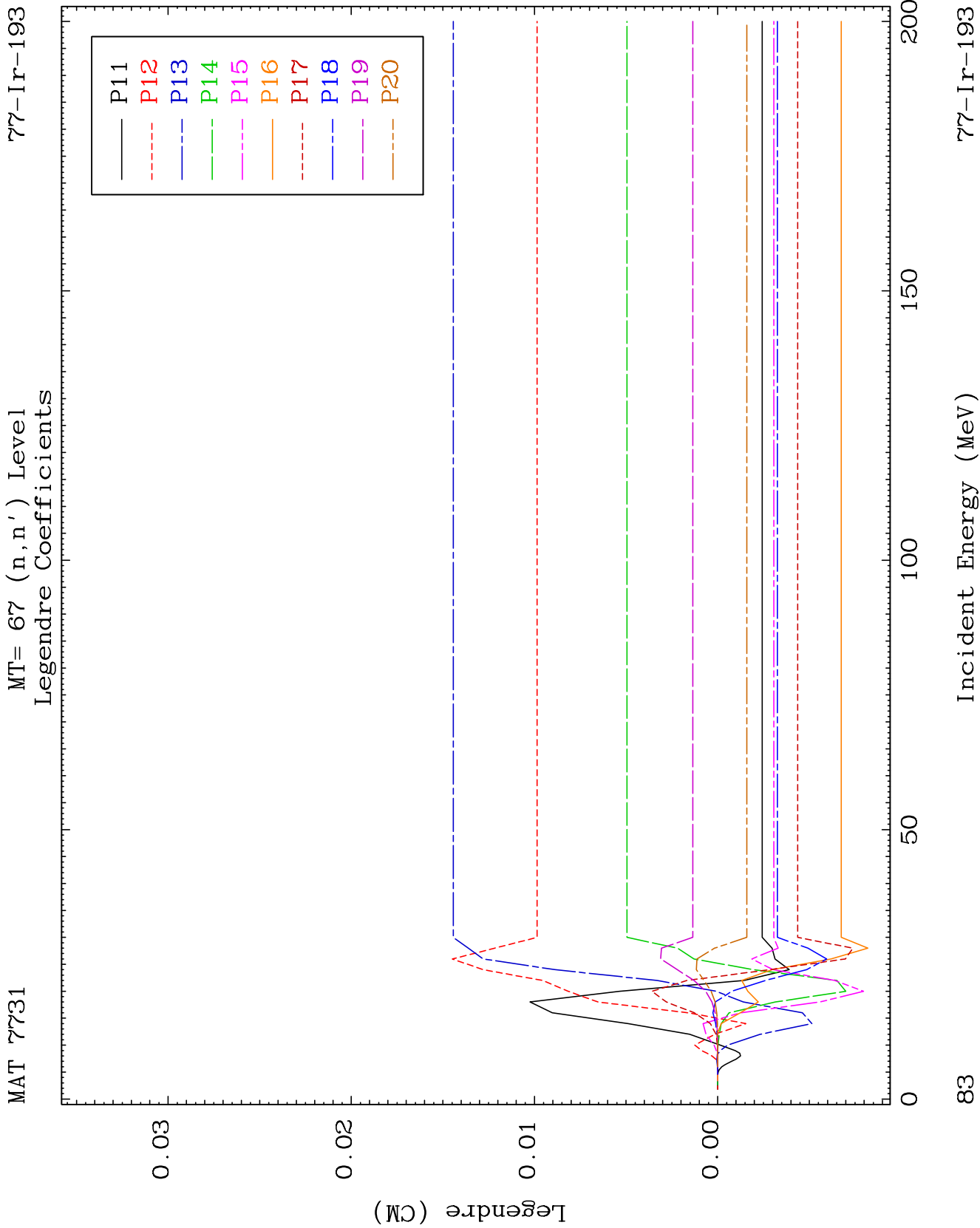
77-Ir-193

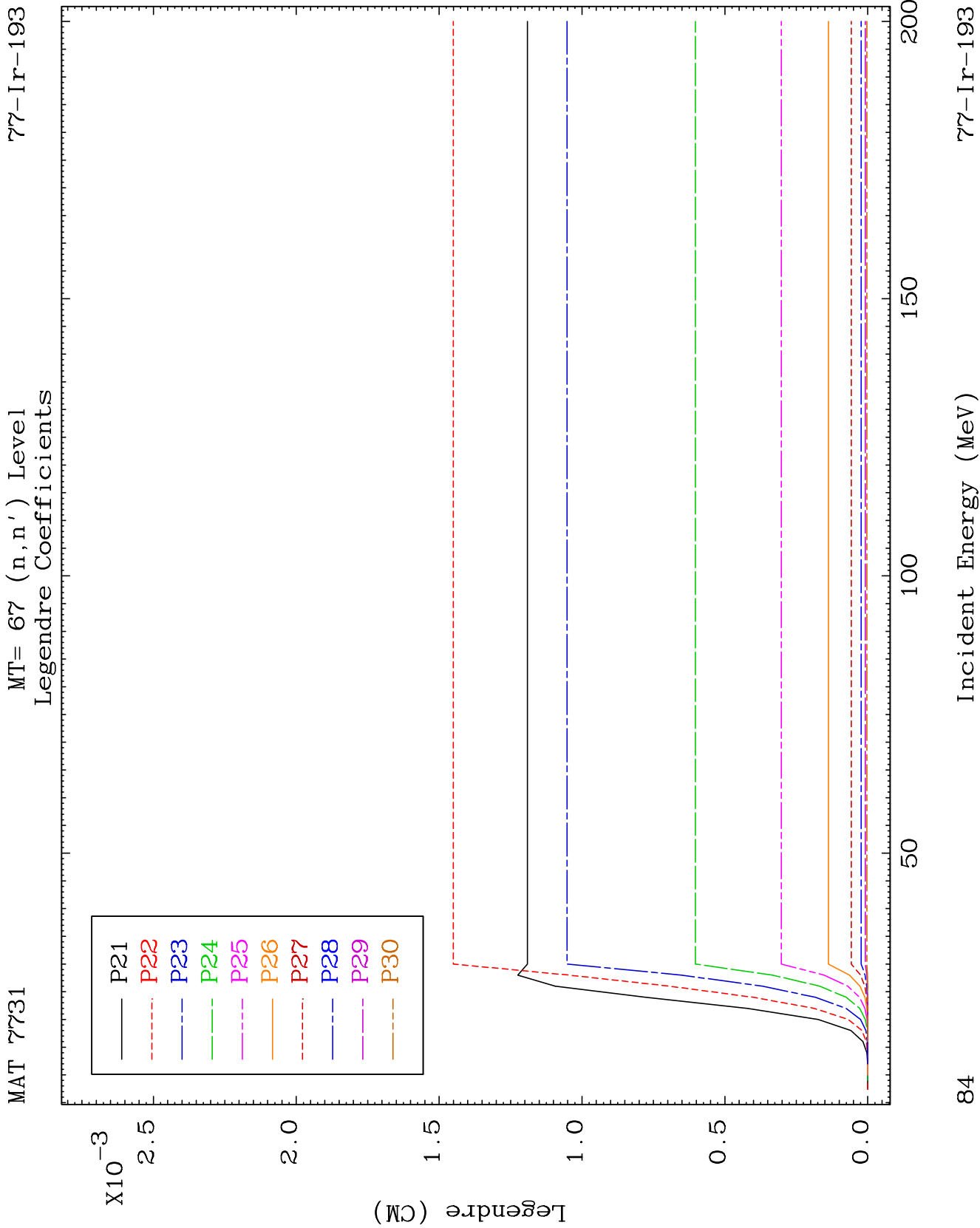


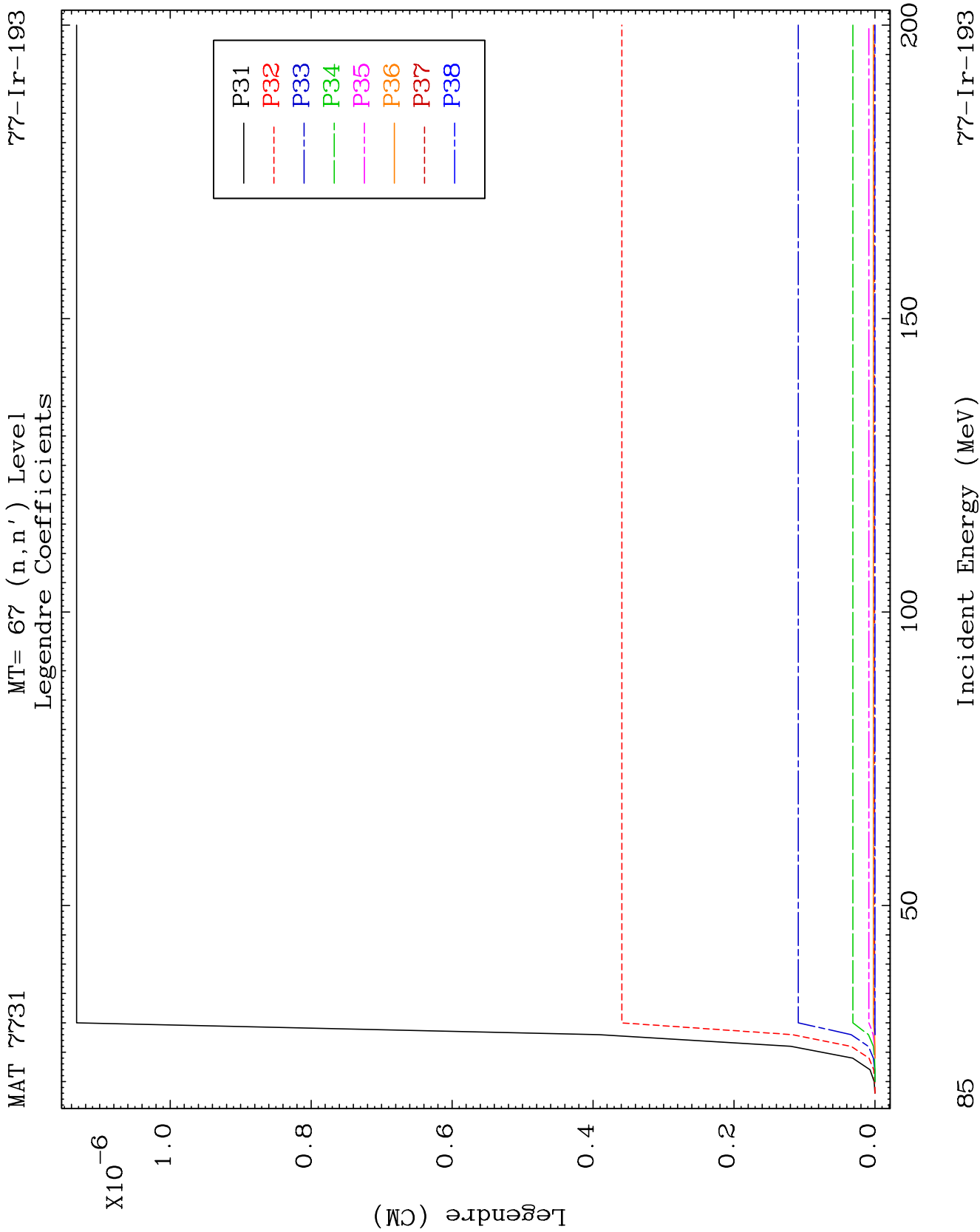




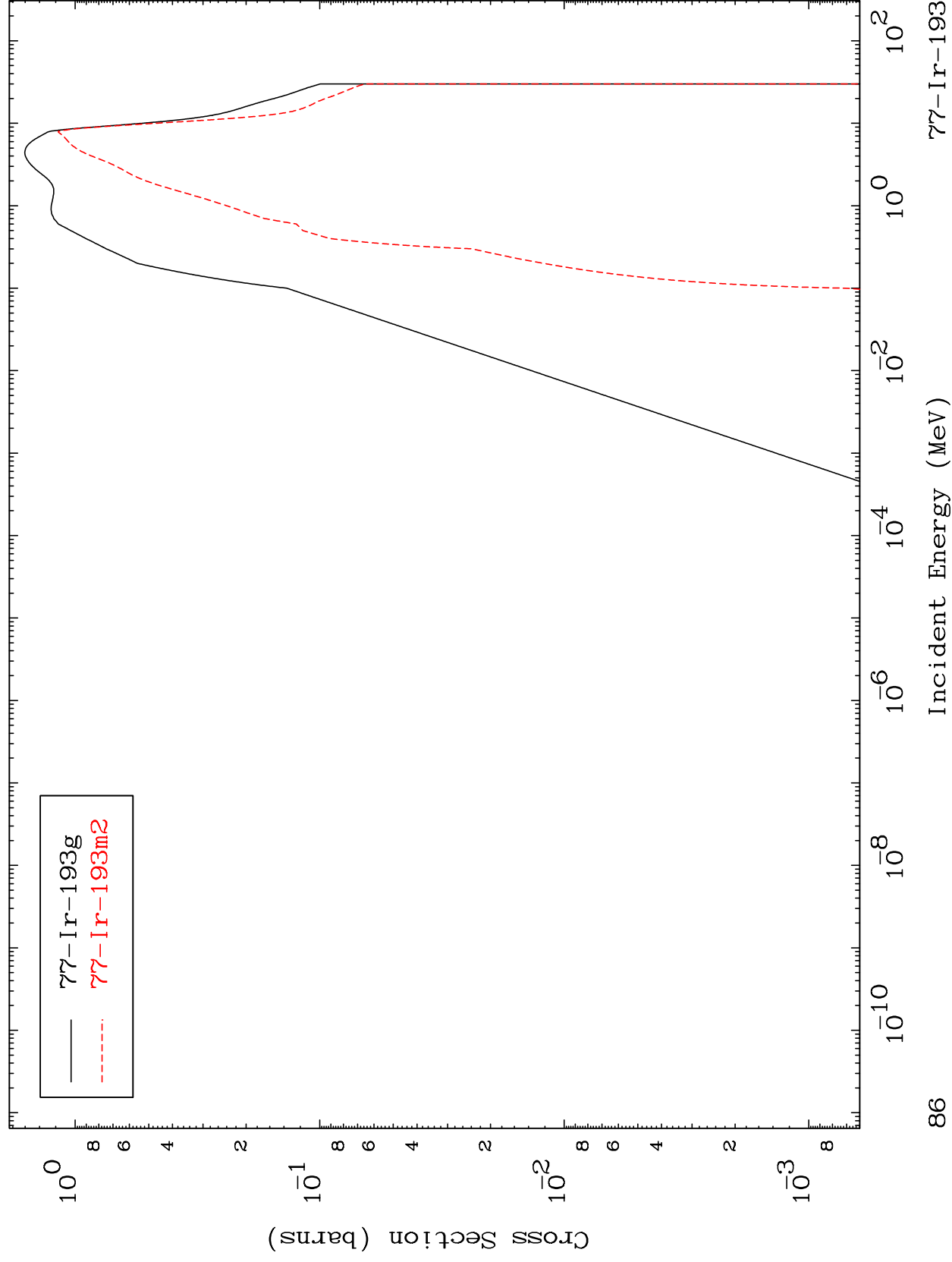








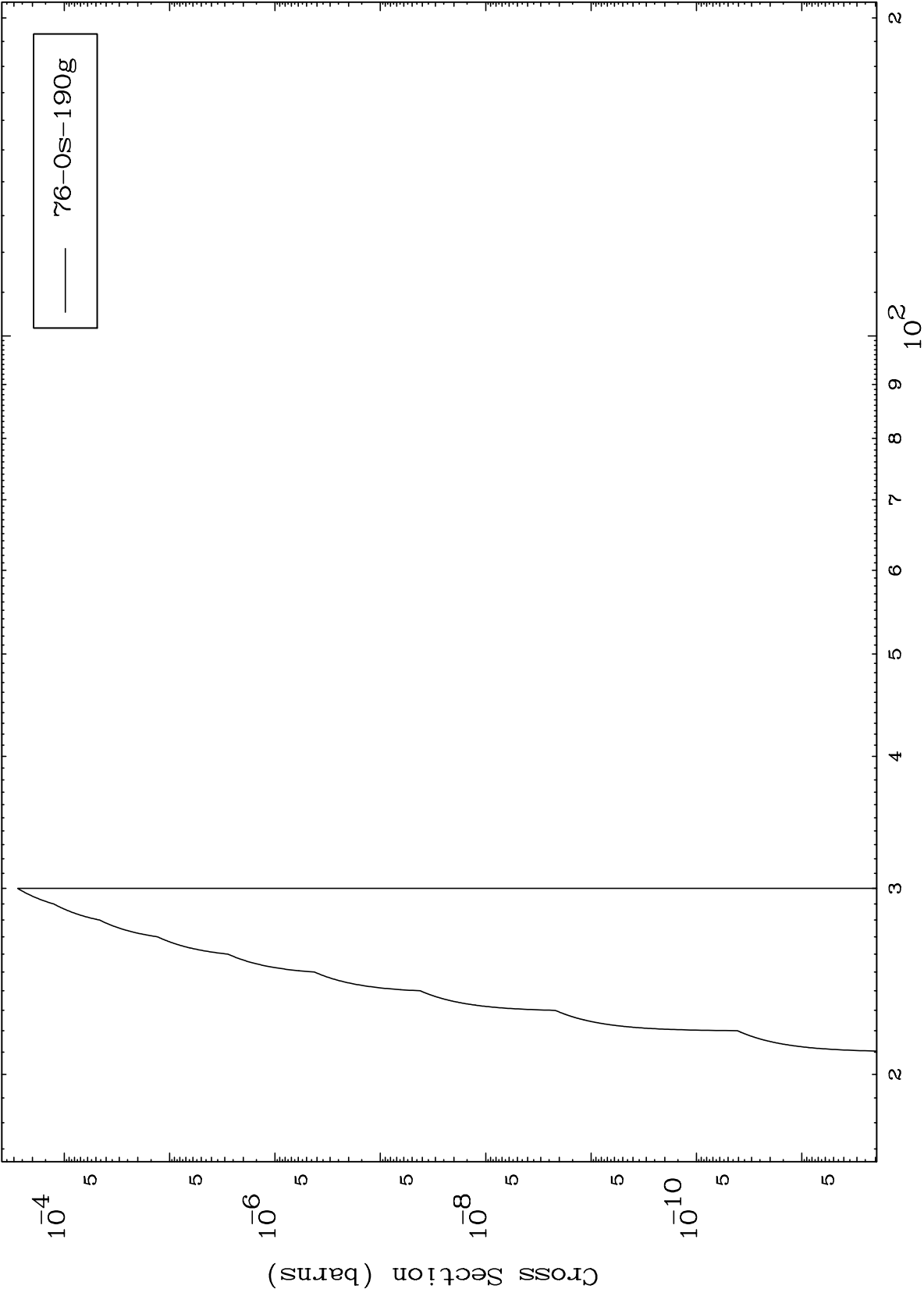
Radionuclide Production Cross Section



MAT 7731

77-Ir-193

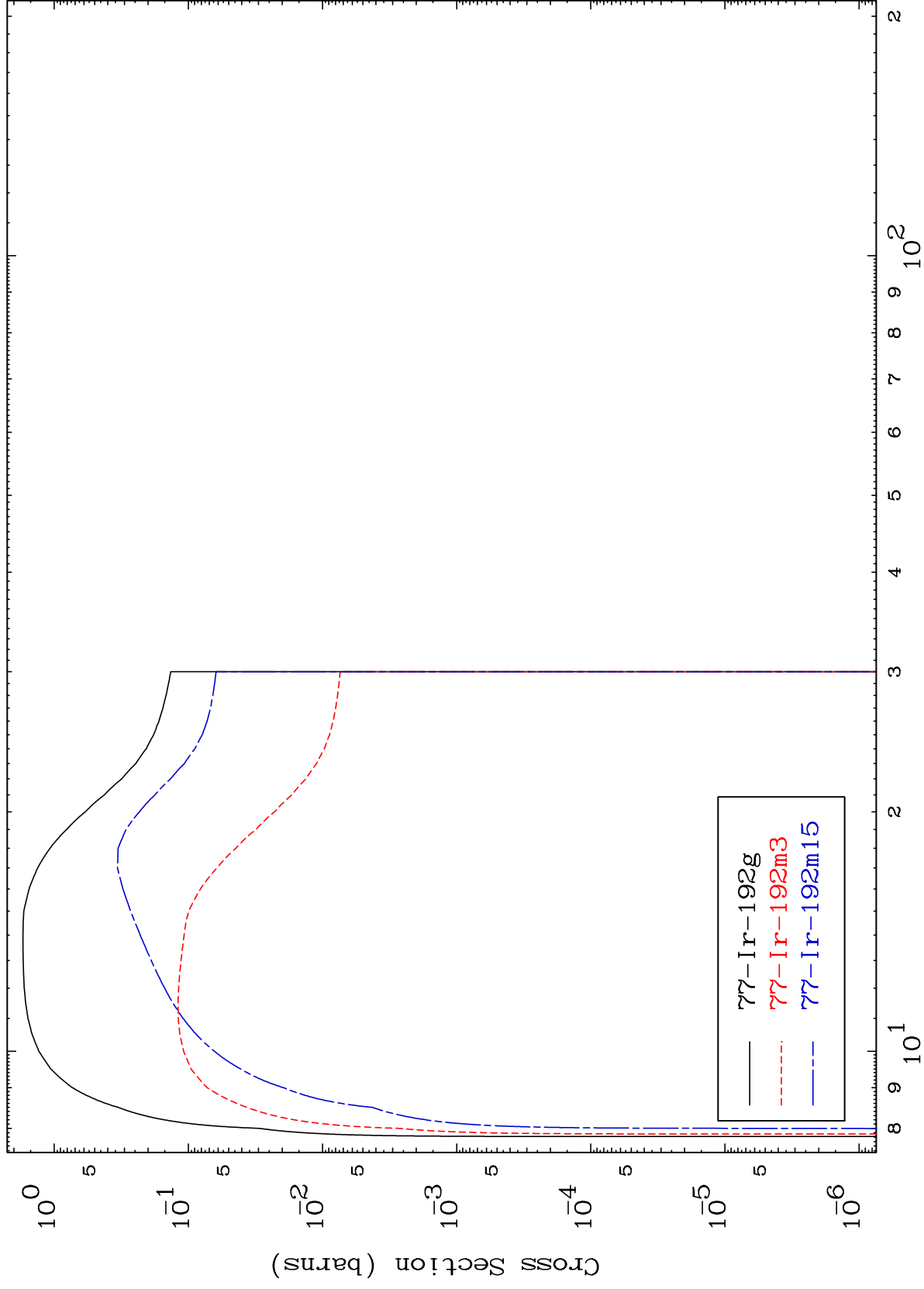
(n,2n) d
Radionuclide Production Cross Section

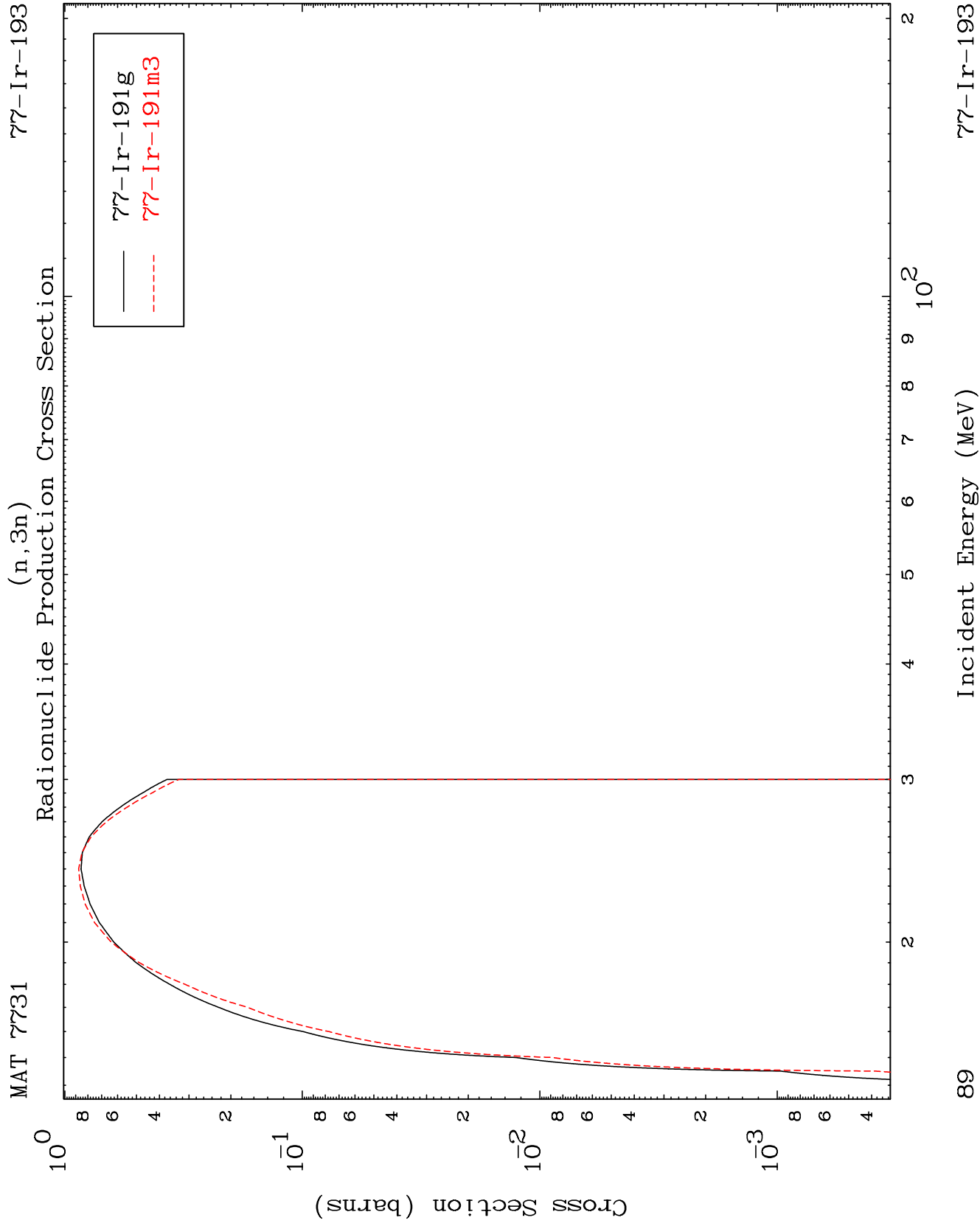


87

Incident Energy (MeV)

77-Ir-193

(n,2n)
Radionuclide Production Cross Section

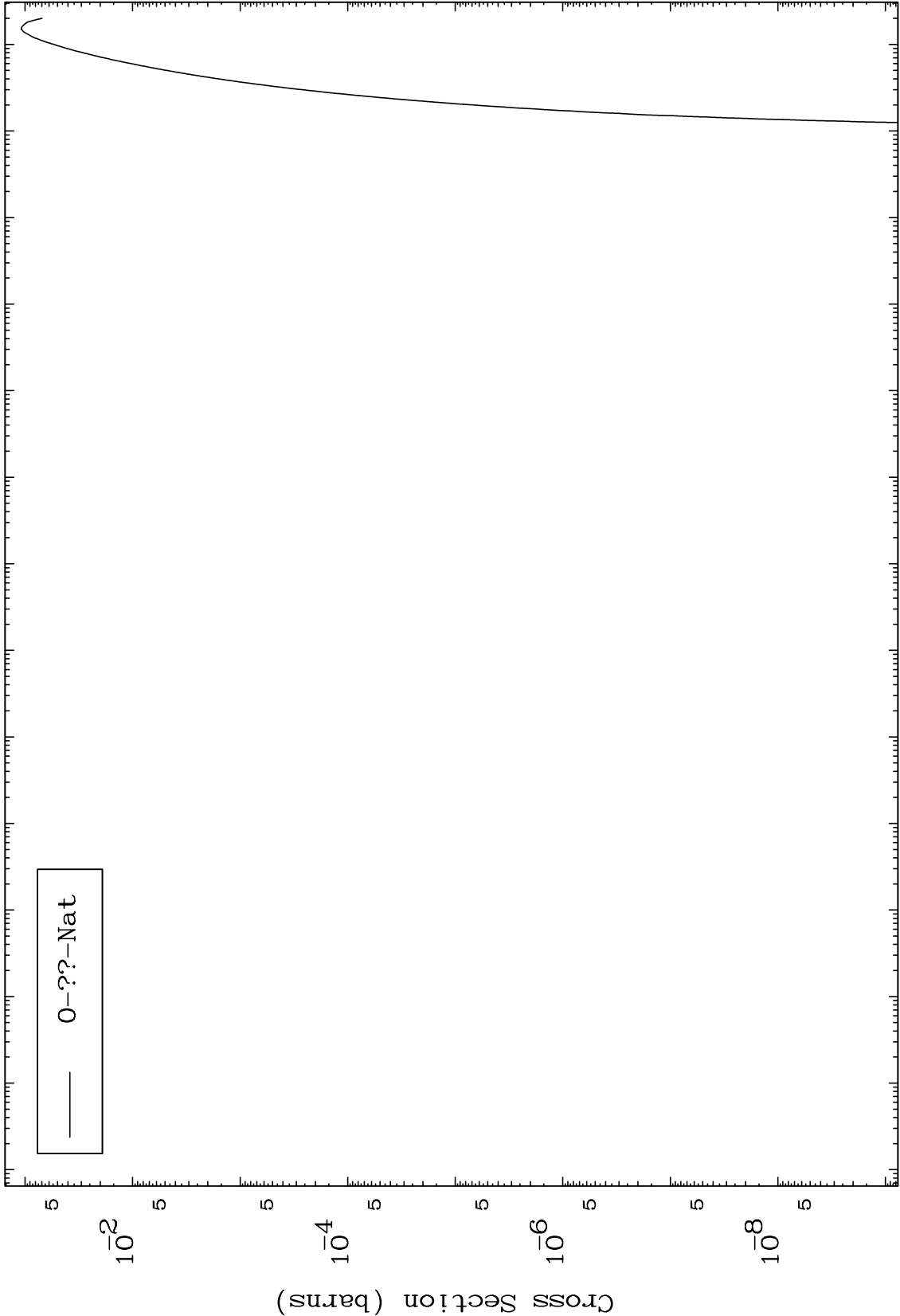


MAT 7731

Fission

⁷⁷Ir-193

Radionuclide Production Cross Section



90

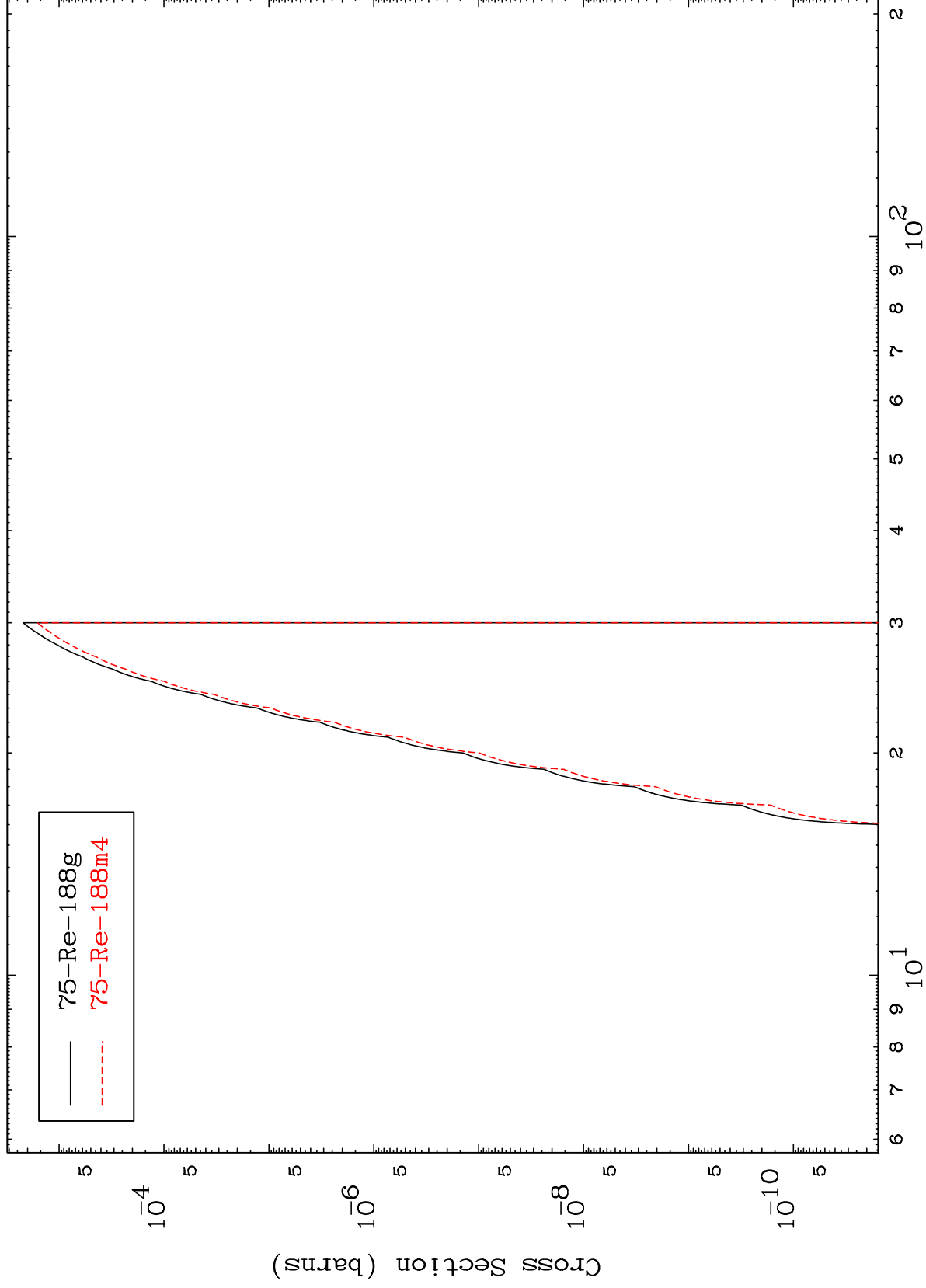
Incident Energy (MeV)

⁷⁷Ir-193

MAT 7731

77-Ir-193

(n,2n) α
Radionuclide Production Cross Section



91

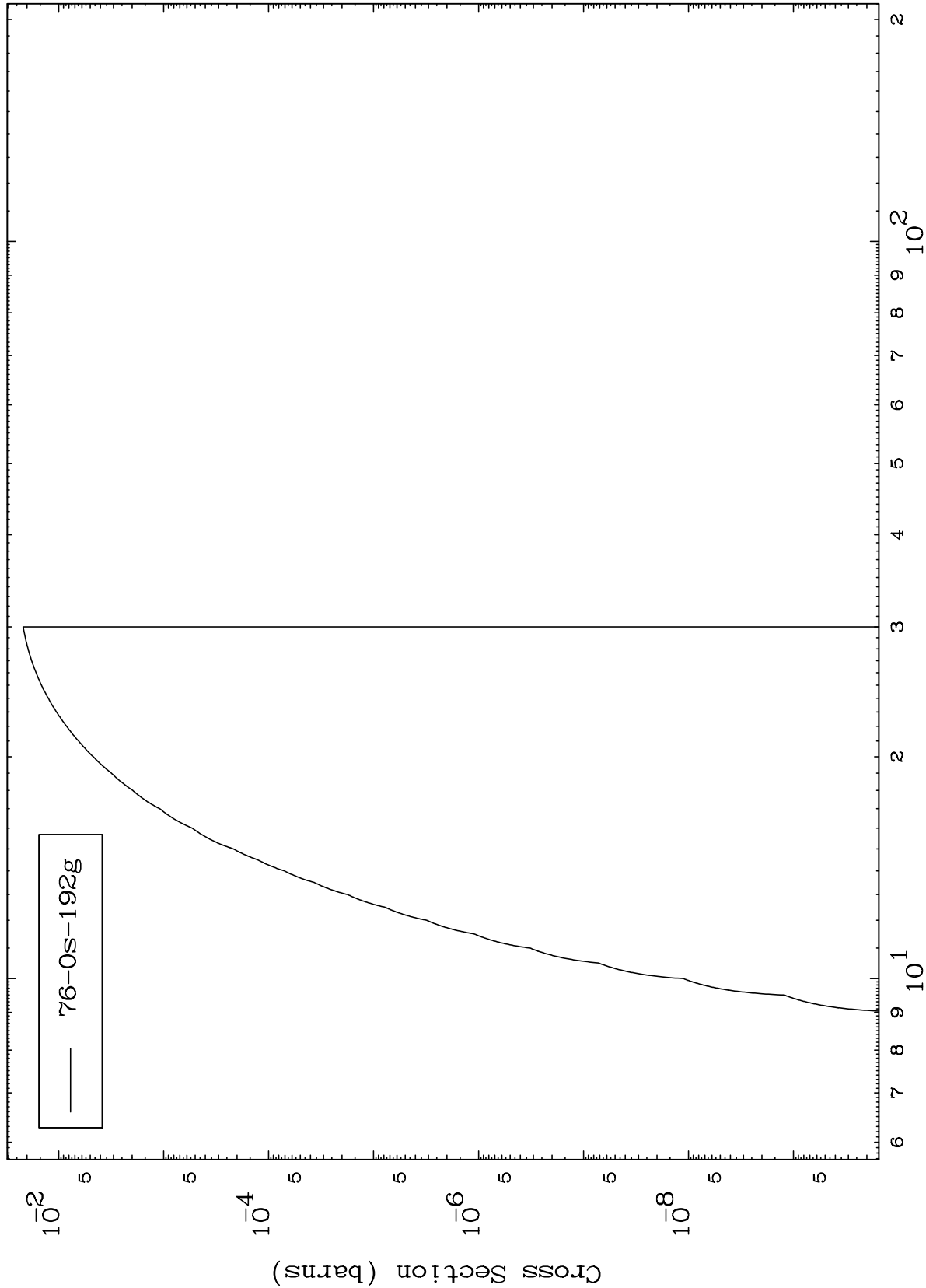
Incident Energy (MeV)

77-Ir-193

MAT 7731

⁷⁷Ir-193

(n,n') p
Radionuclide Production Cross Section

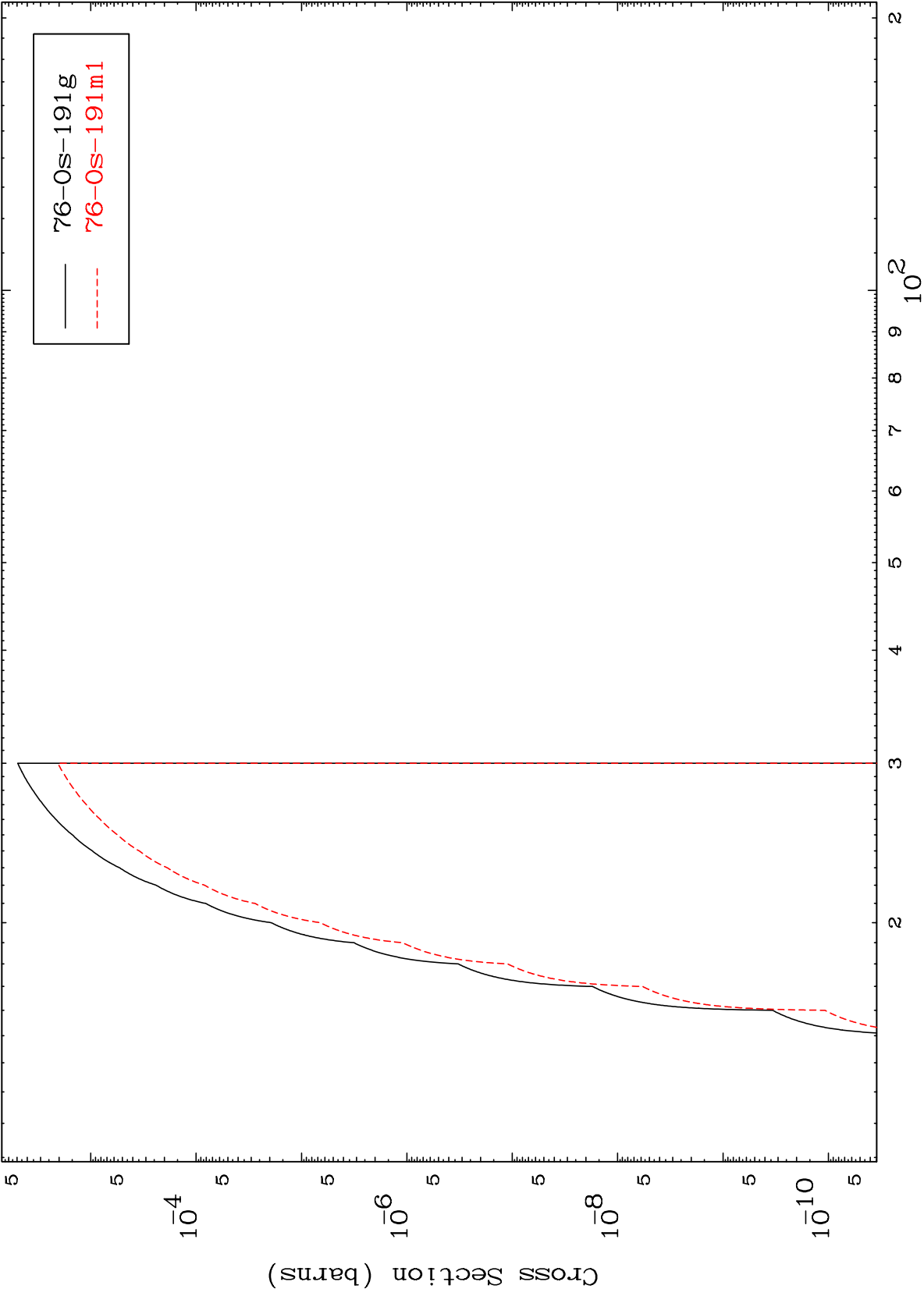


92

Incident Energy (MeV)

⁷⁷Ir-193

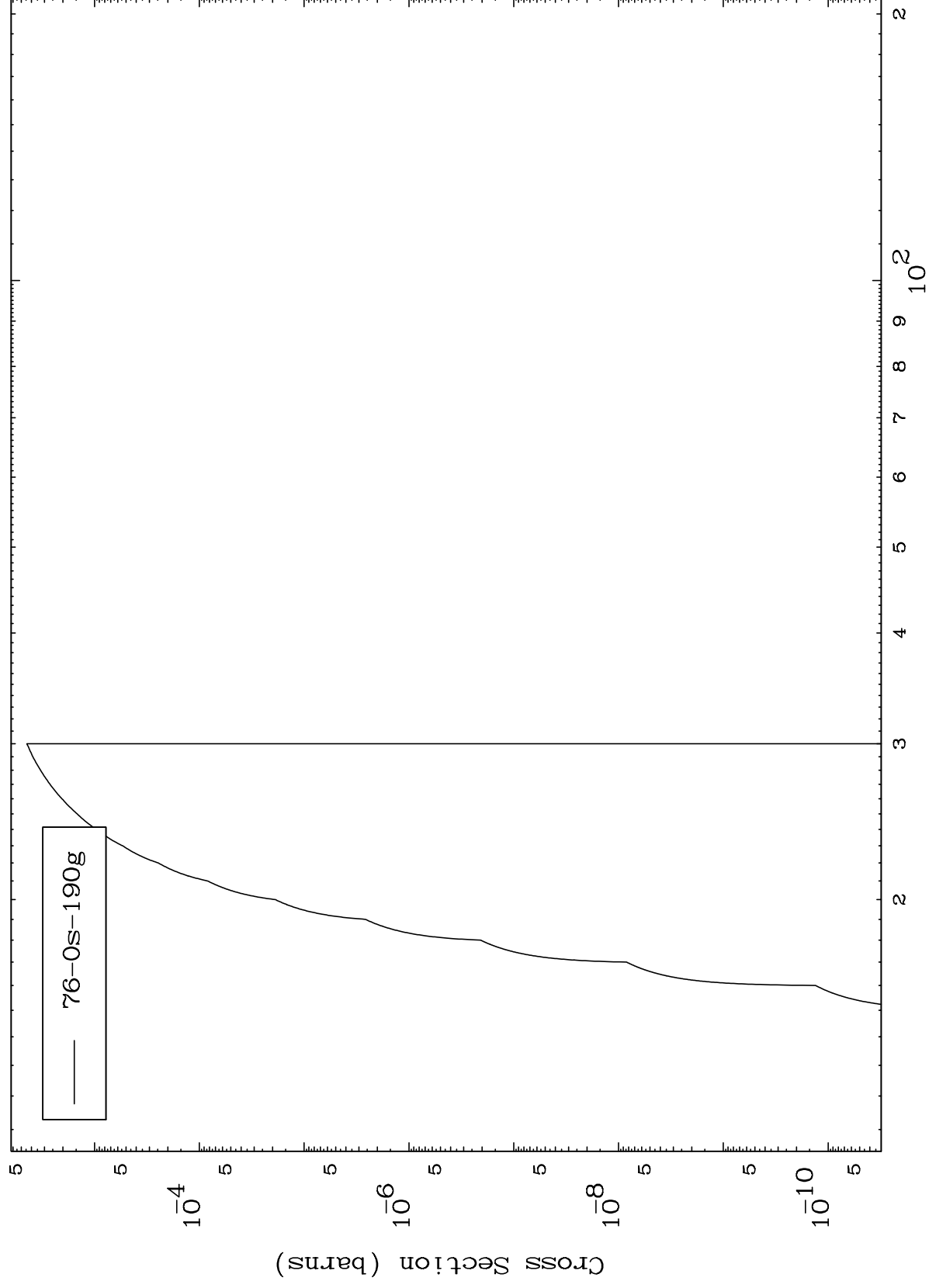
Radionuclide Production Cross Section



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77-Ir-193

(n,n') t
Radionuclide Production Cross Section



94

Incident Energy (MeV)

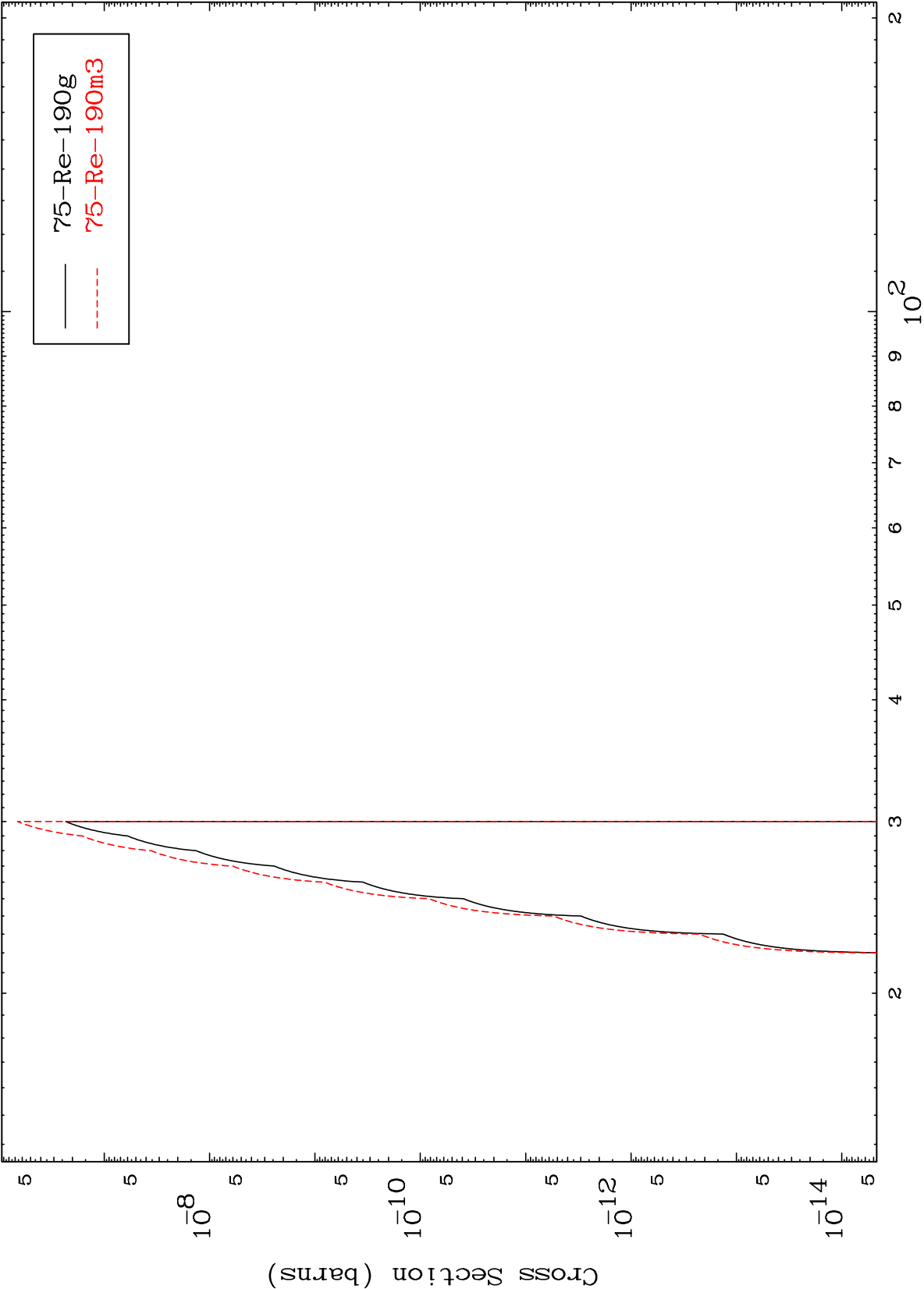
77-Ir-193

MAT 7731

(n,n') He-3

77-Ir-193

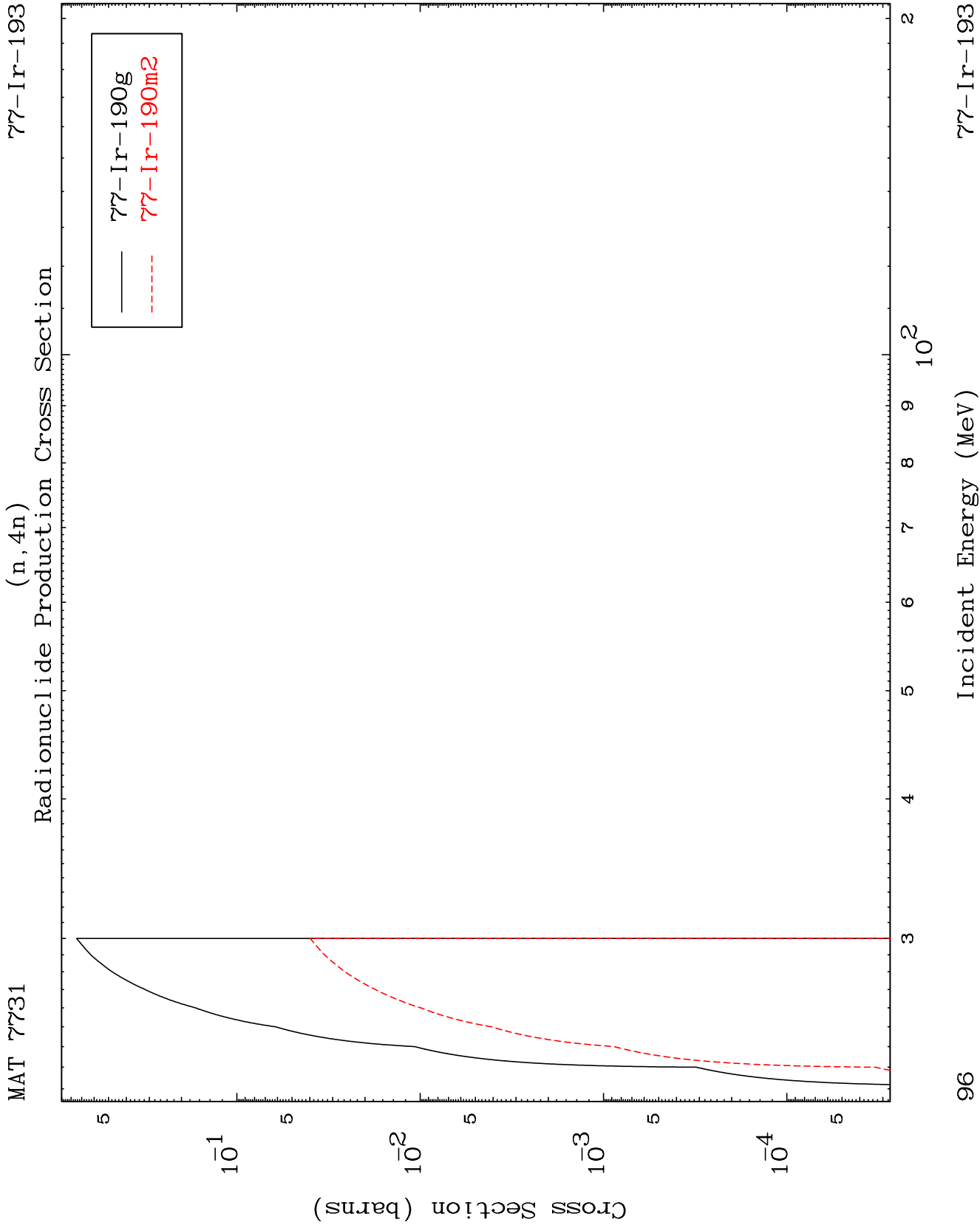
Radionuclide Production Cross Section

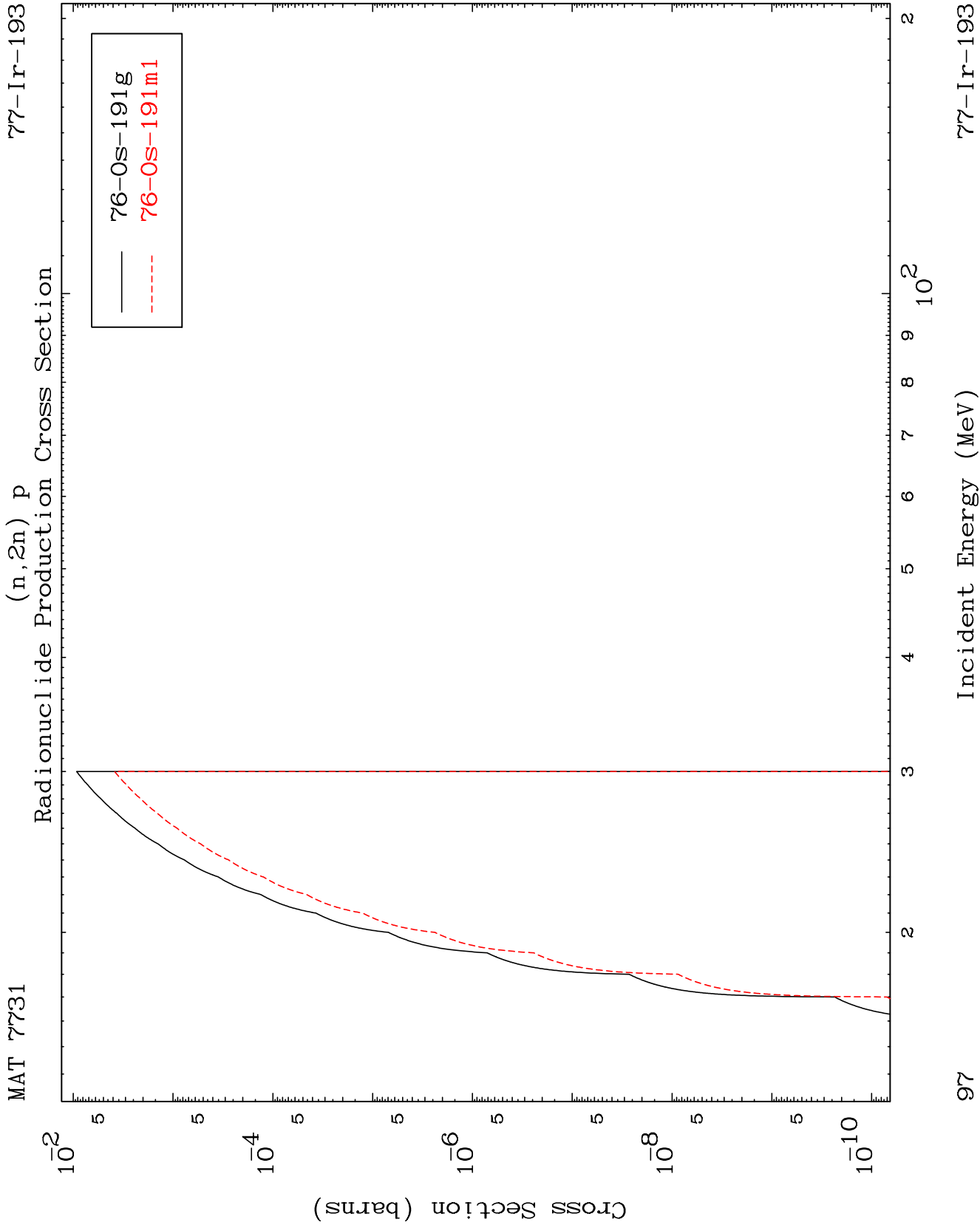


95

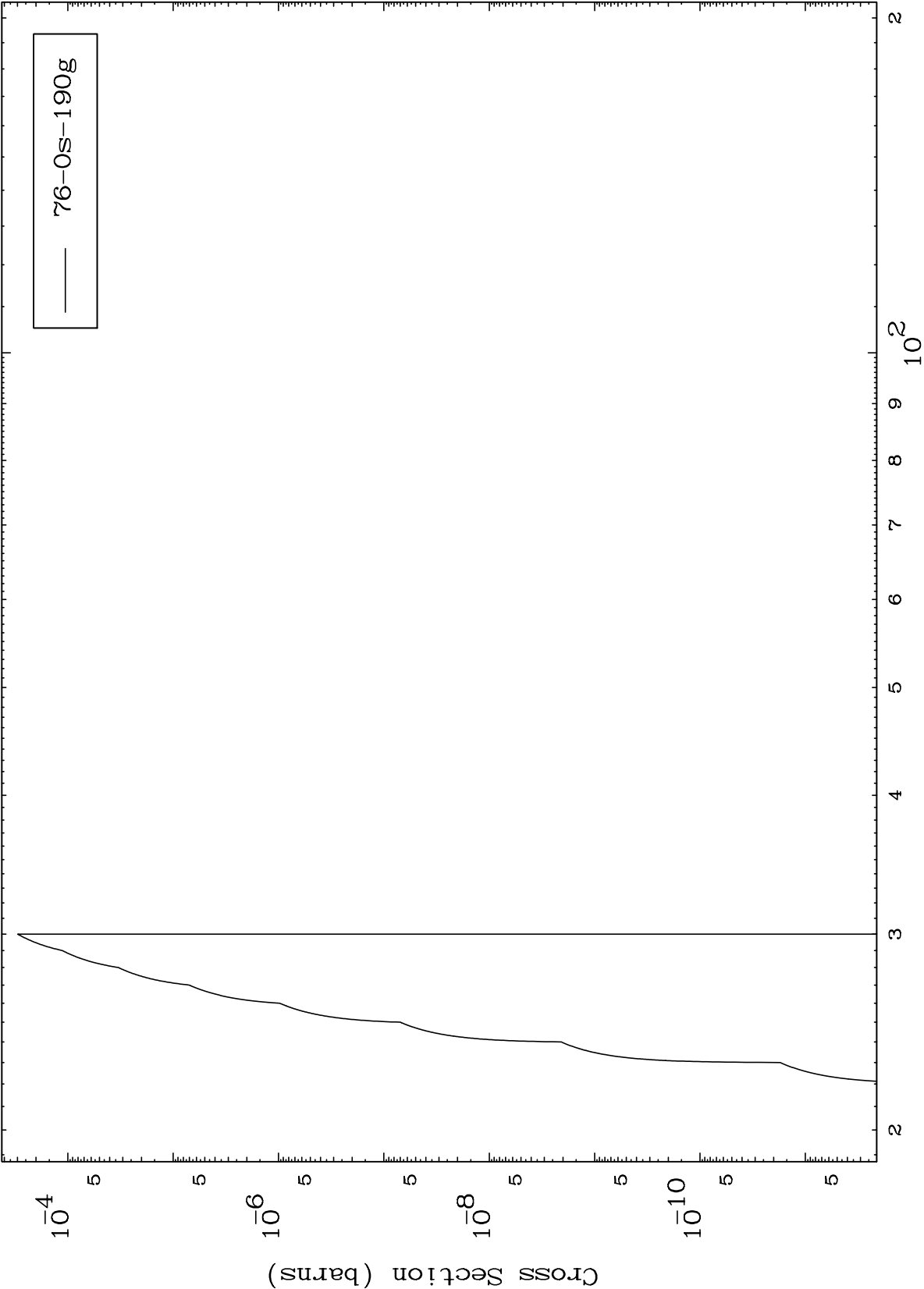
Incident Energy (MeV)

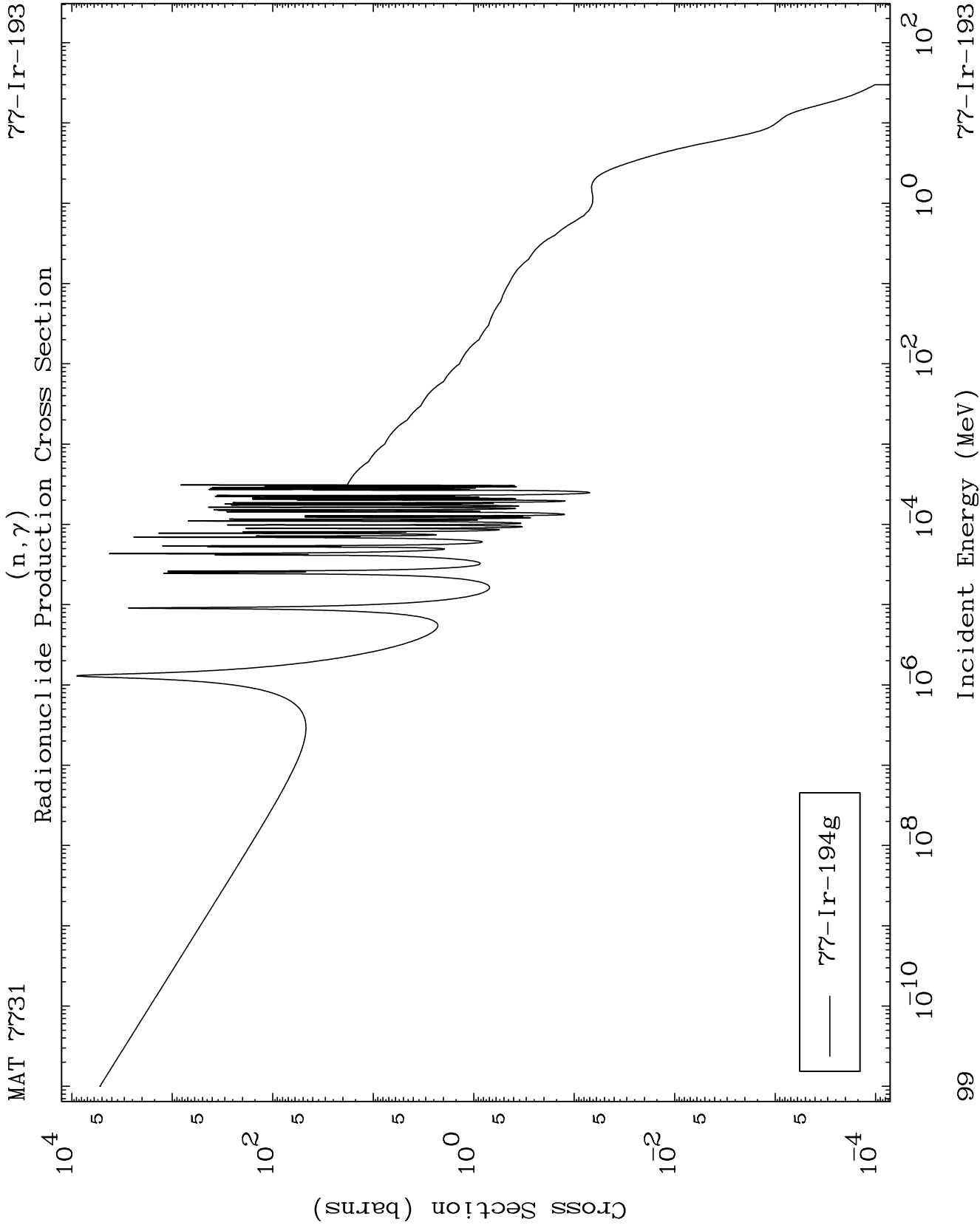
77-Ir-193

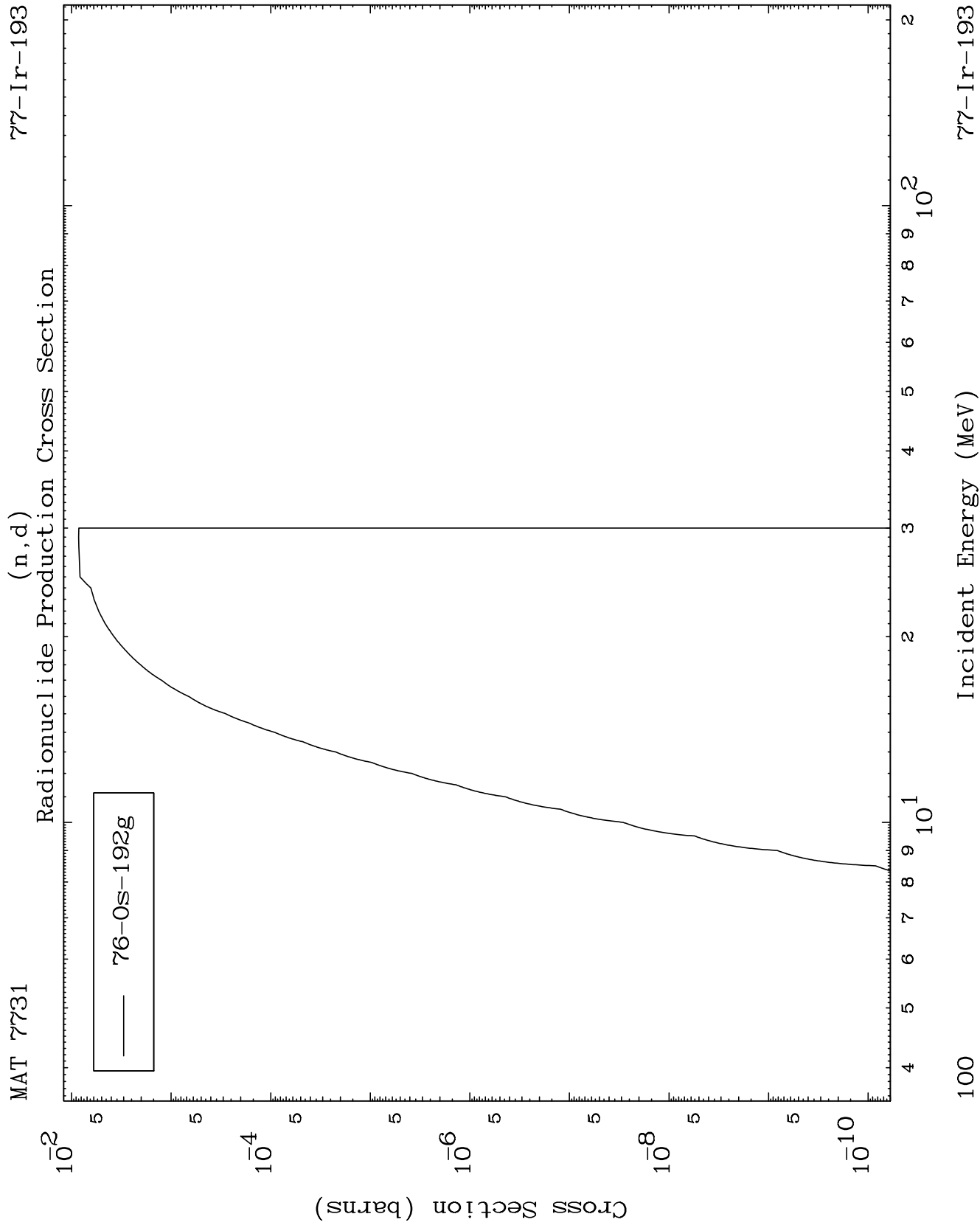


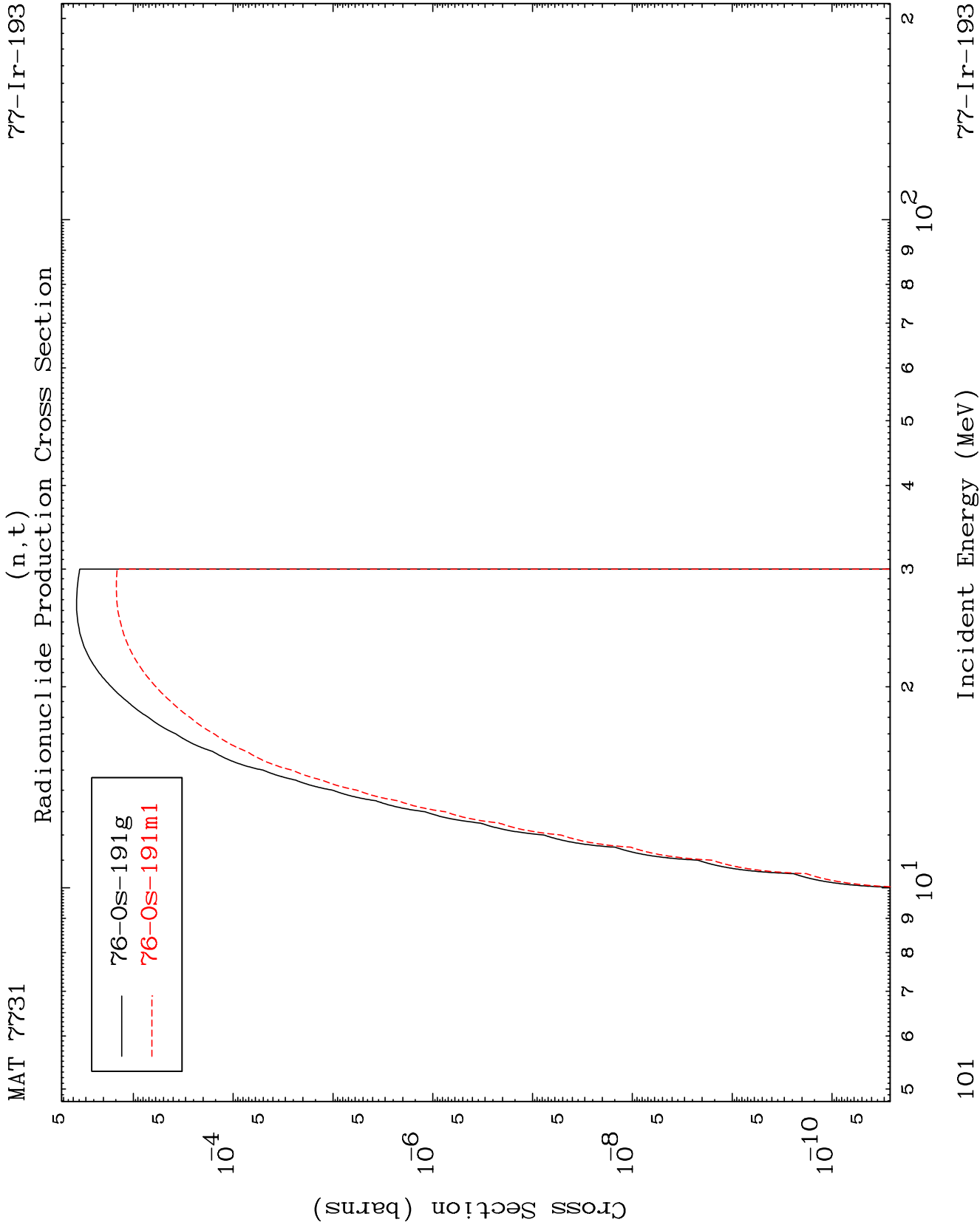


(n,3n) p
Radionuclide Production Cross Section







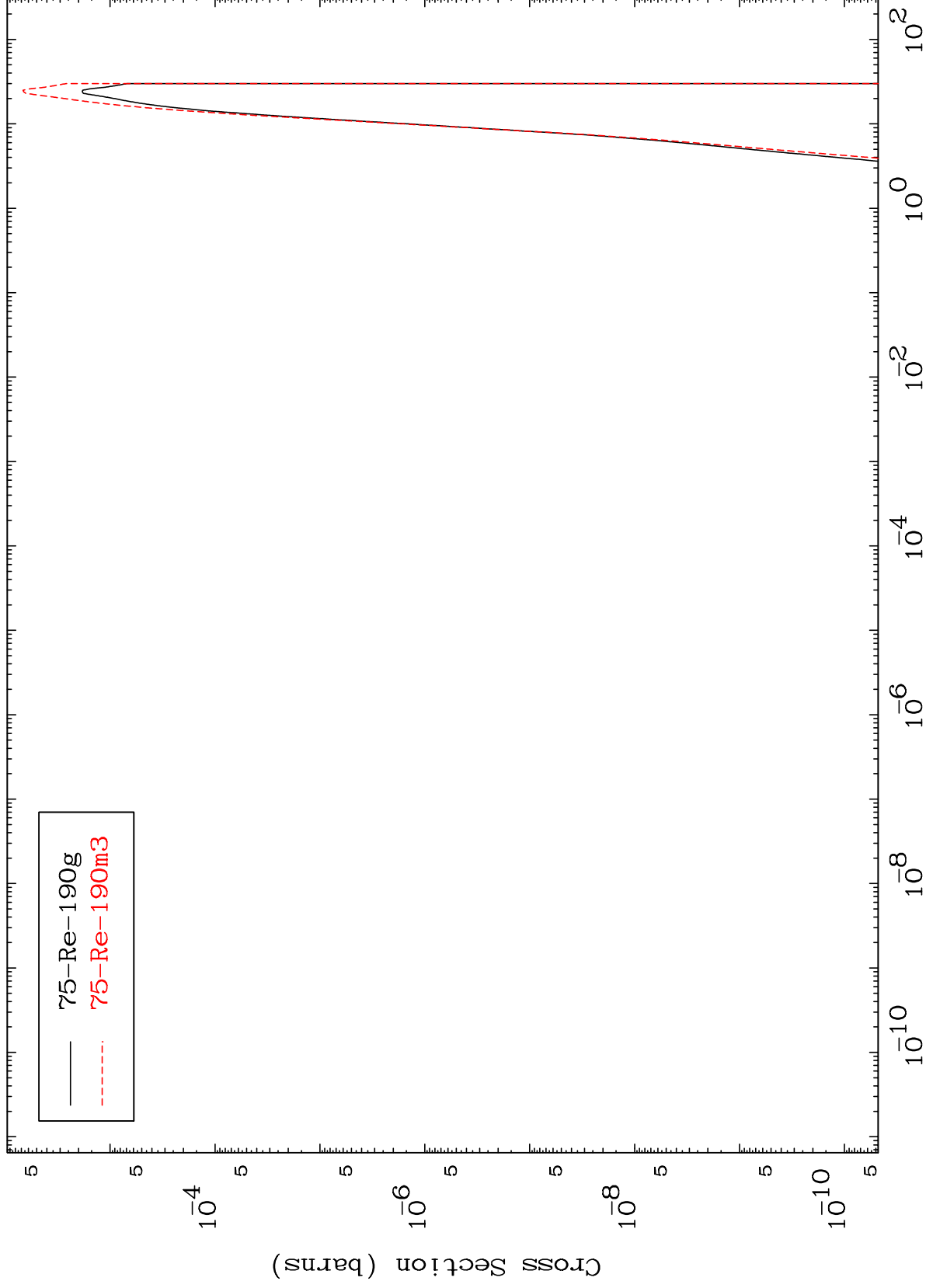


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(n, α)

⁷⁷Ir-193

Radionuclide Production Cross Section



102

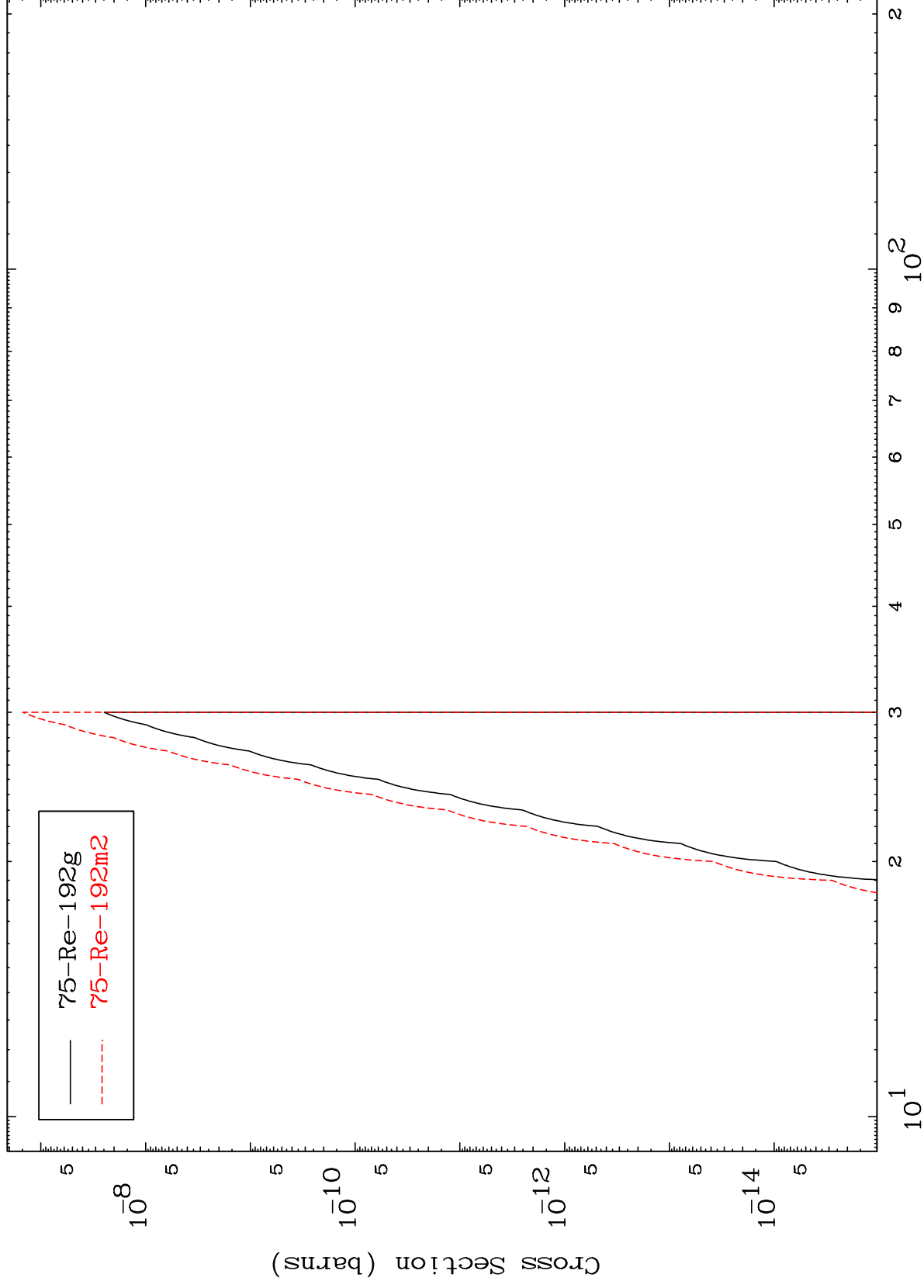
Incident Energy (MeV)

⁷⁷Ir-193

MAT 7731

77-Ir-193

Radionuclide Production Cross Section
(n,2p)



77-Ir-193

Incident Energy (MeV)