

Program EVALPLOT  
(Version 2021-1)

by

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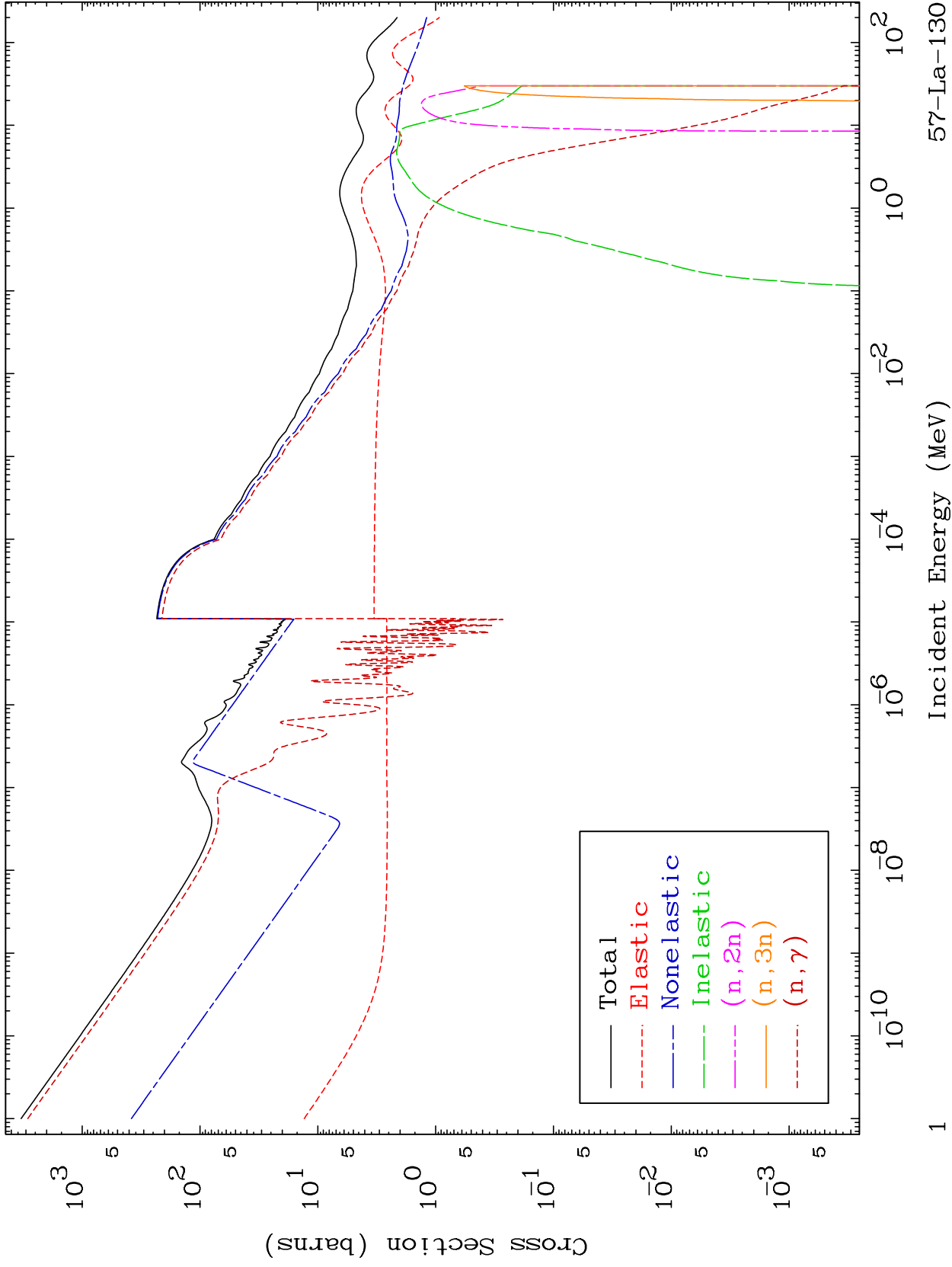
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Press Mouse Button to Start

MAT 5701

Neutron Major  
293 Kelvin Cross Sections

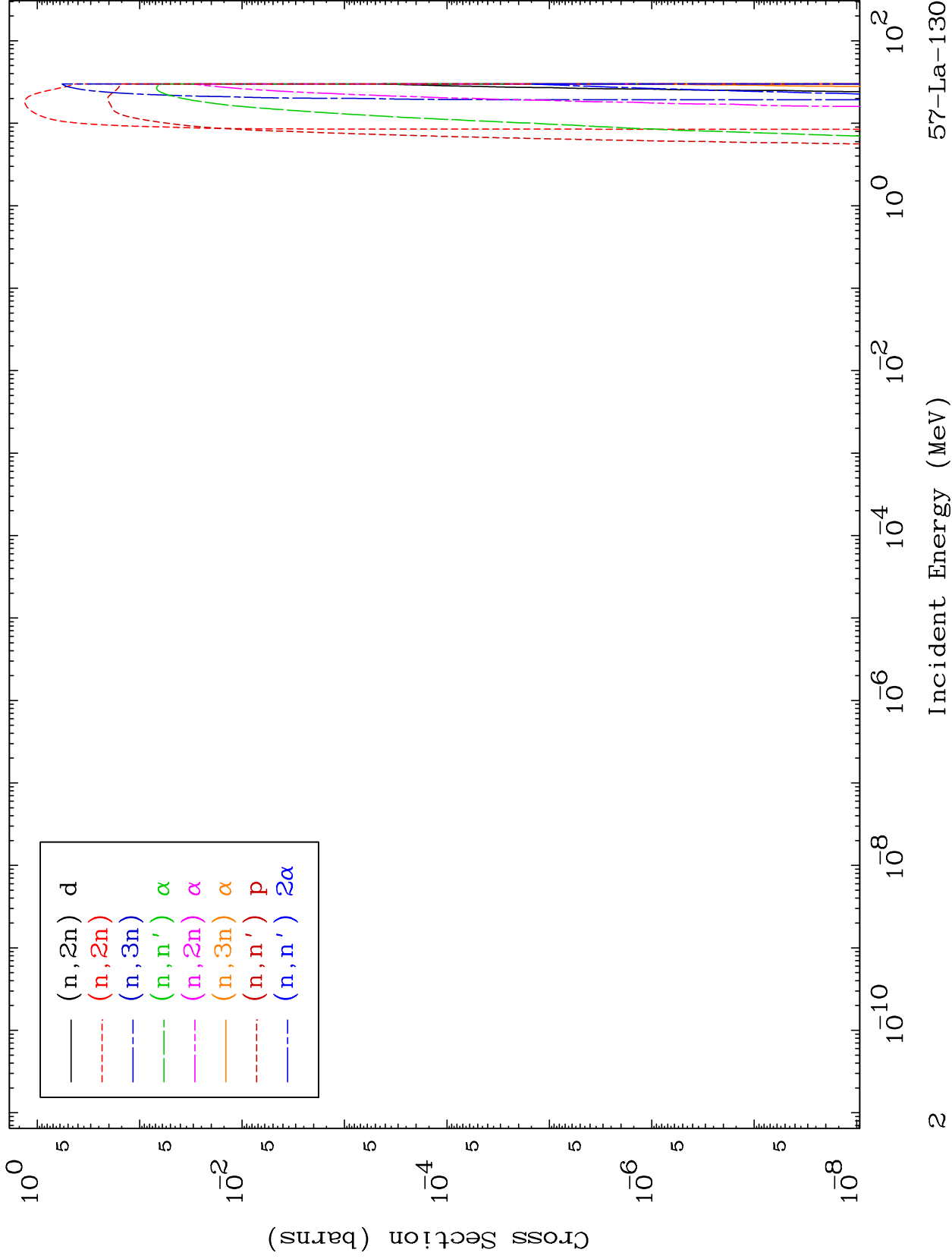
57-La-130



MAT 5701

Neutron Absorption  
293 Kelvin Cross Sections

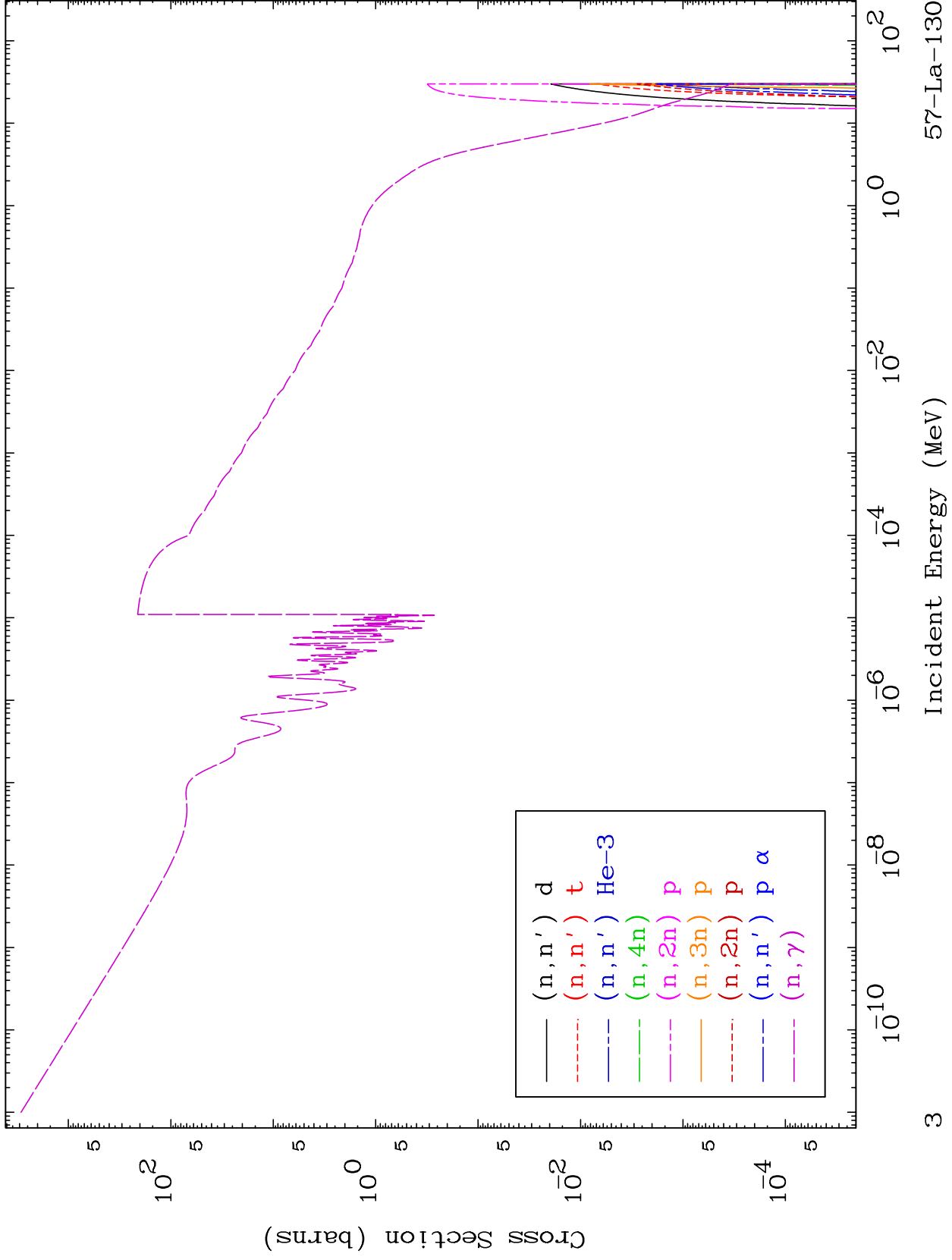
57-La-130



MAT 5701

Neutron Absorption  
293 Kelvin Cross Sections

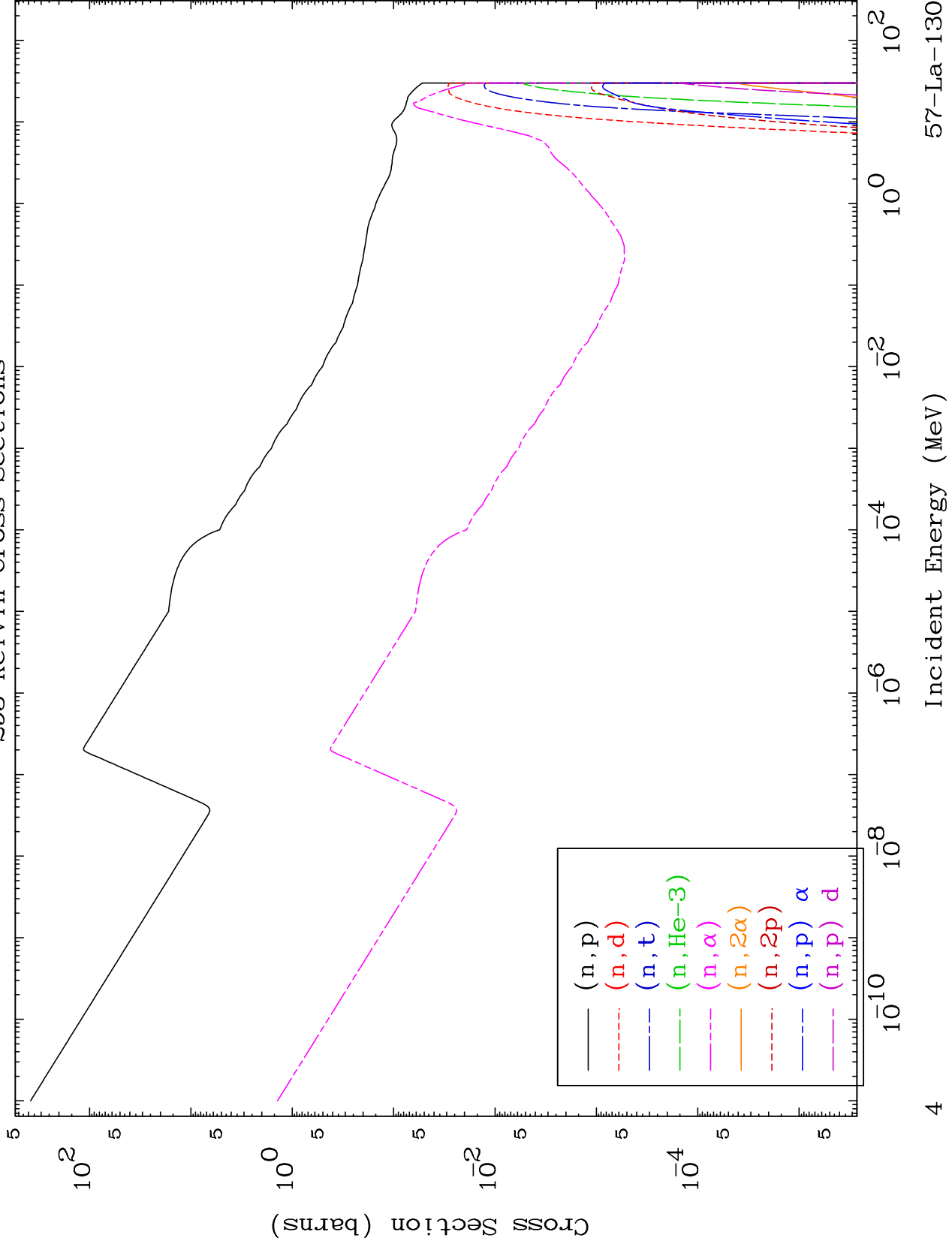
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MAT 5701

Neutron Absorption  
293 Kelvin Cross Sections

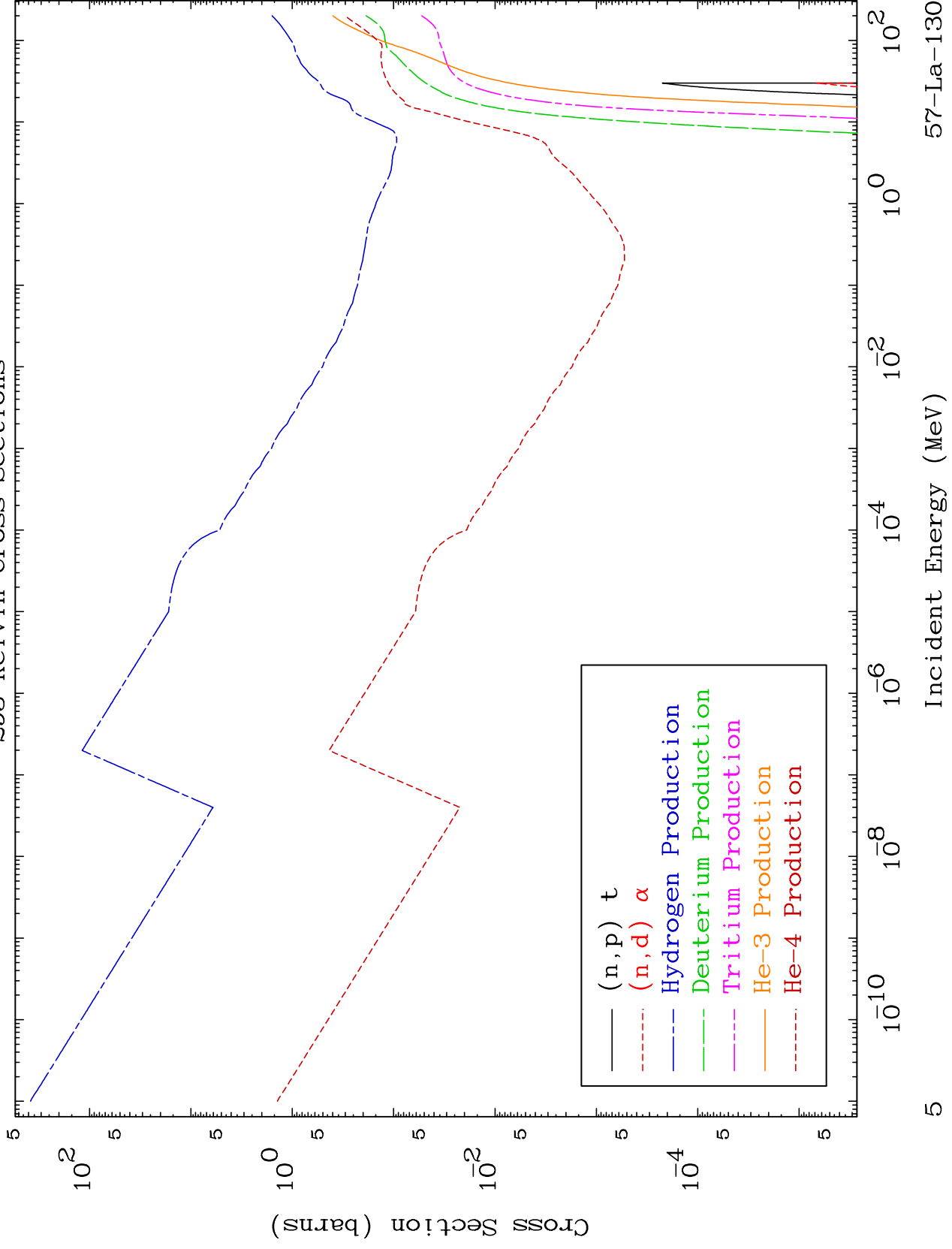
57-La-130



MAT 5701

Neutron Absorption  
293 Kelvin Cross Sections

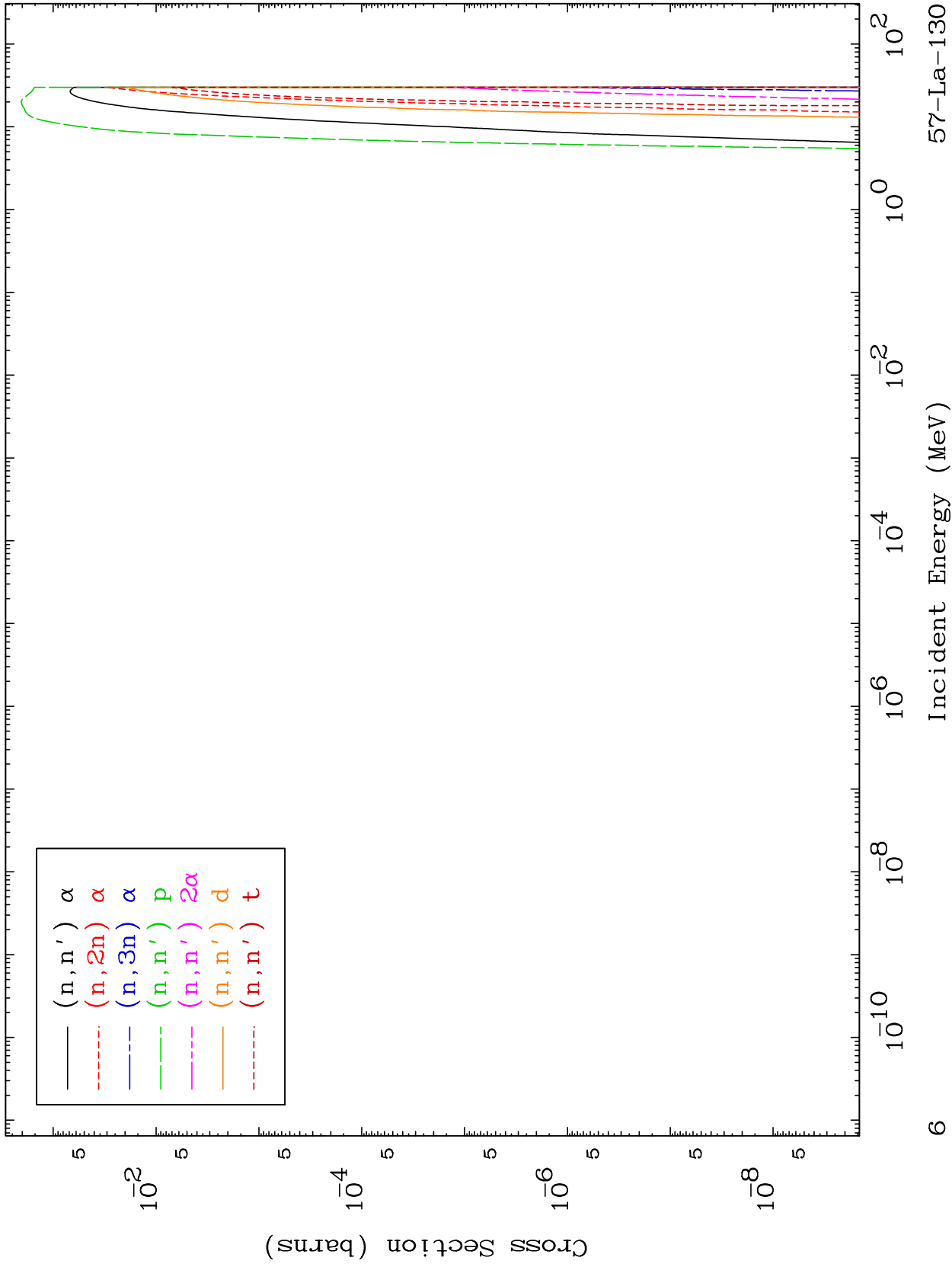
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MAT 5701

Charged Particle  
293 Kelvin Cross Sections

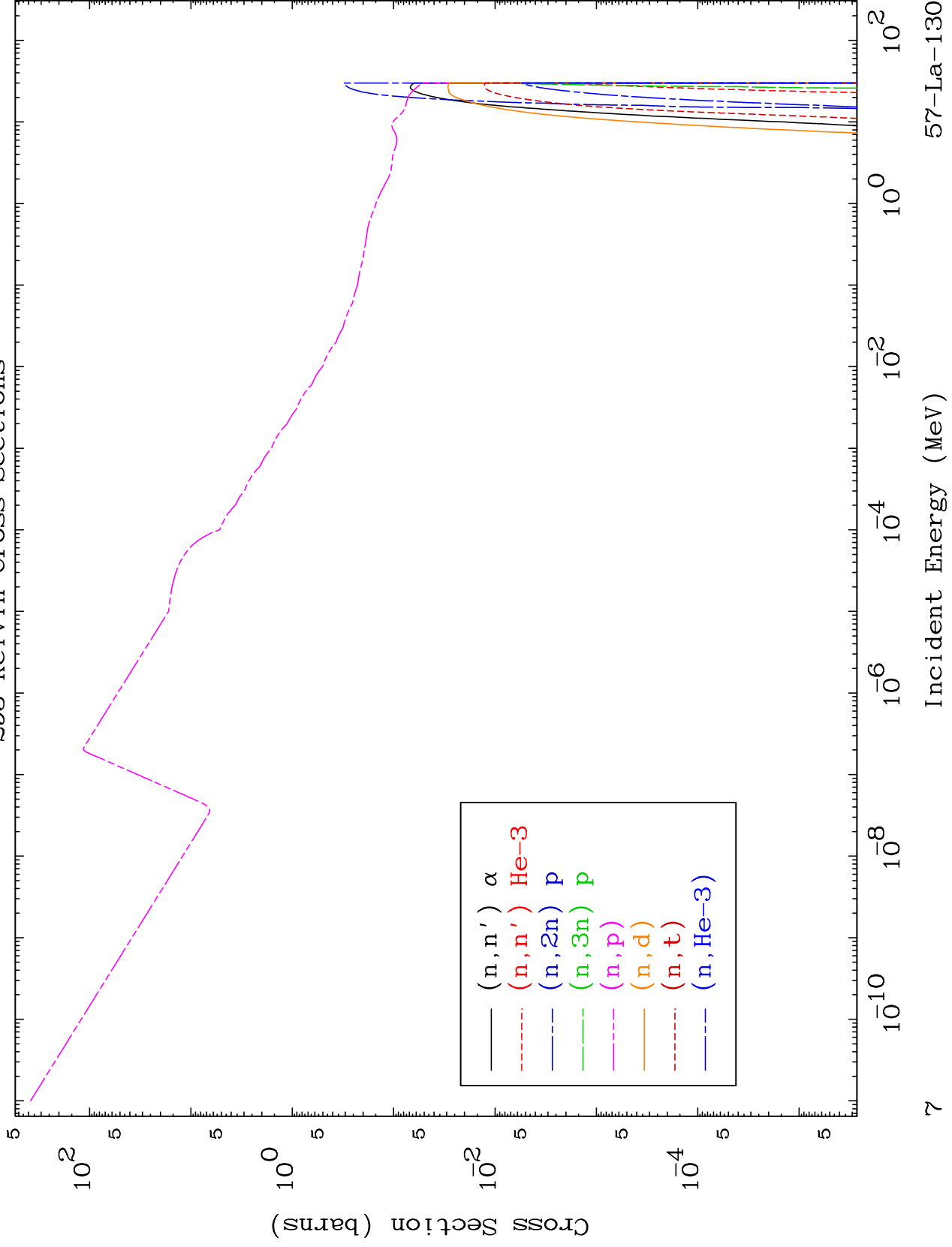
57-La-130



MAT 5701

Charged Particle  
293 Kelvin Cross Sections

57-La-130

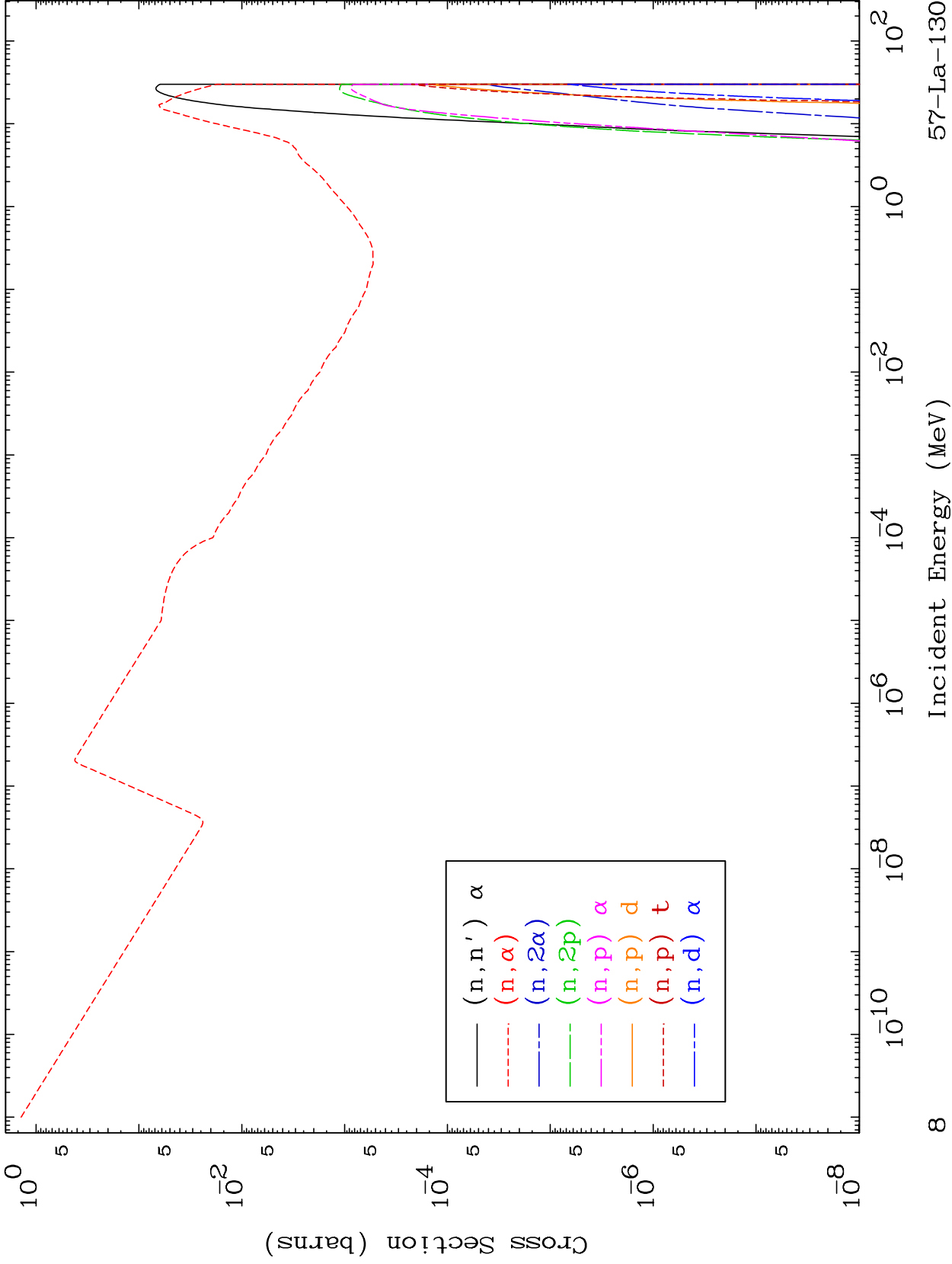




MAT 5701

Charged Particle  
293 Kelvin Cross Sections

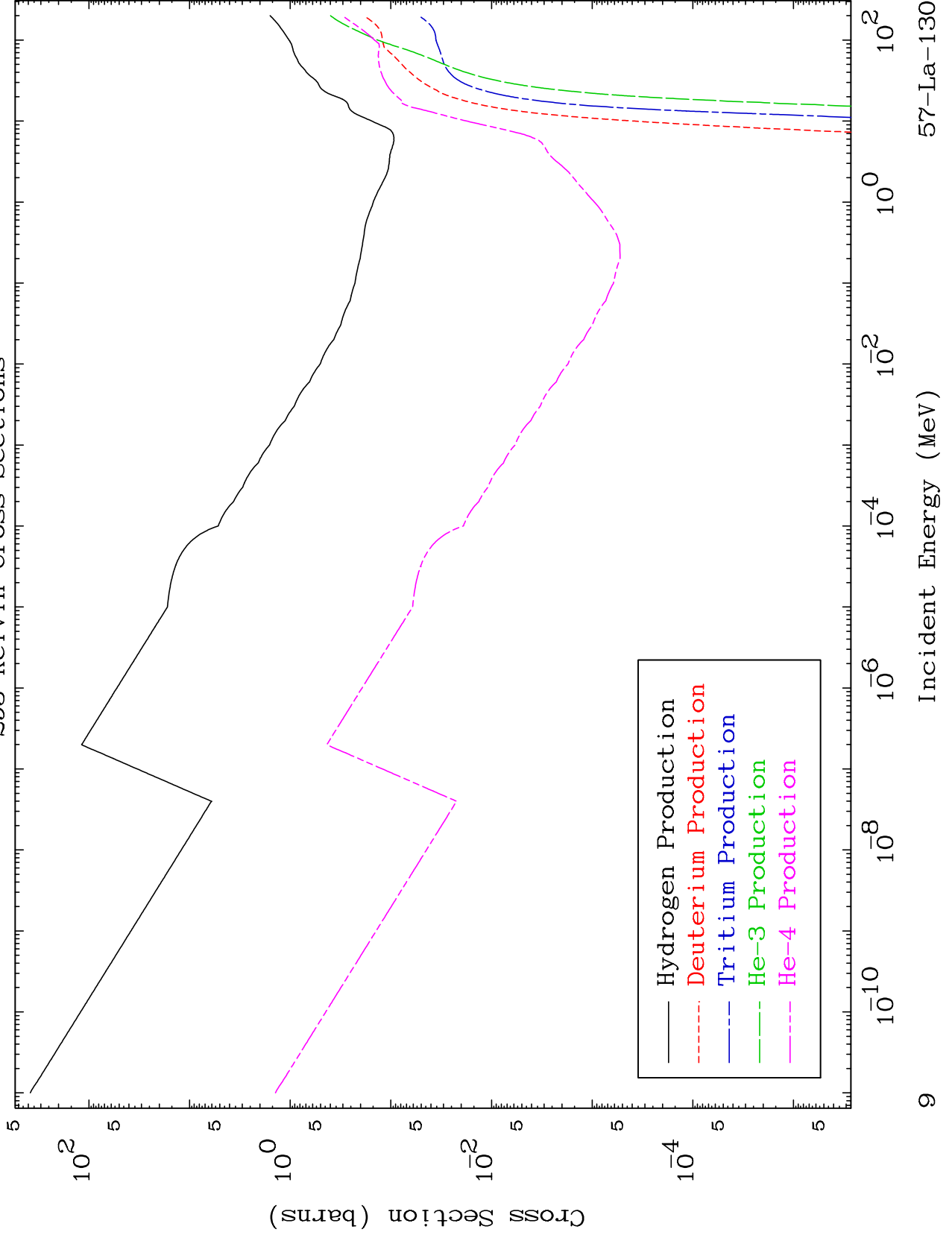
57-La-130



MAT 5701

Particle Production  
293 Kelvin Cross Sections

57-La-130

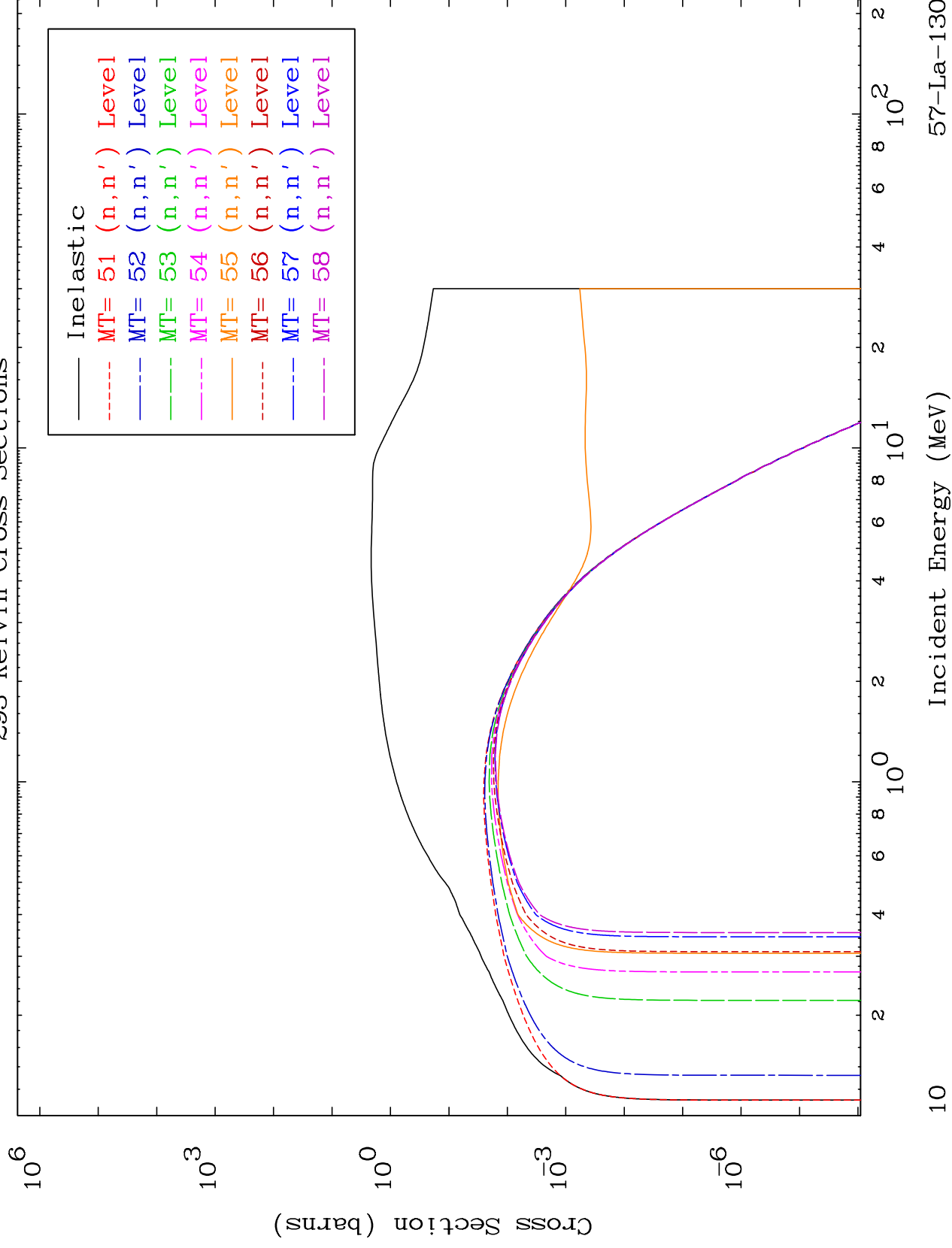


MAT 5701

(n,n') Levels

57-La-130

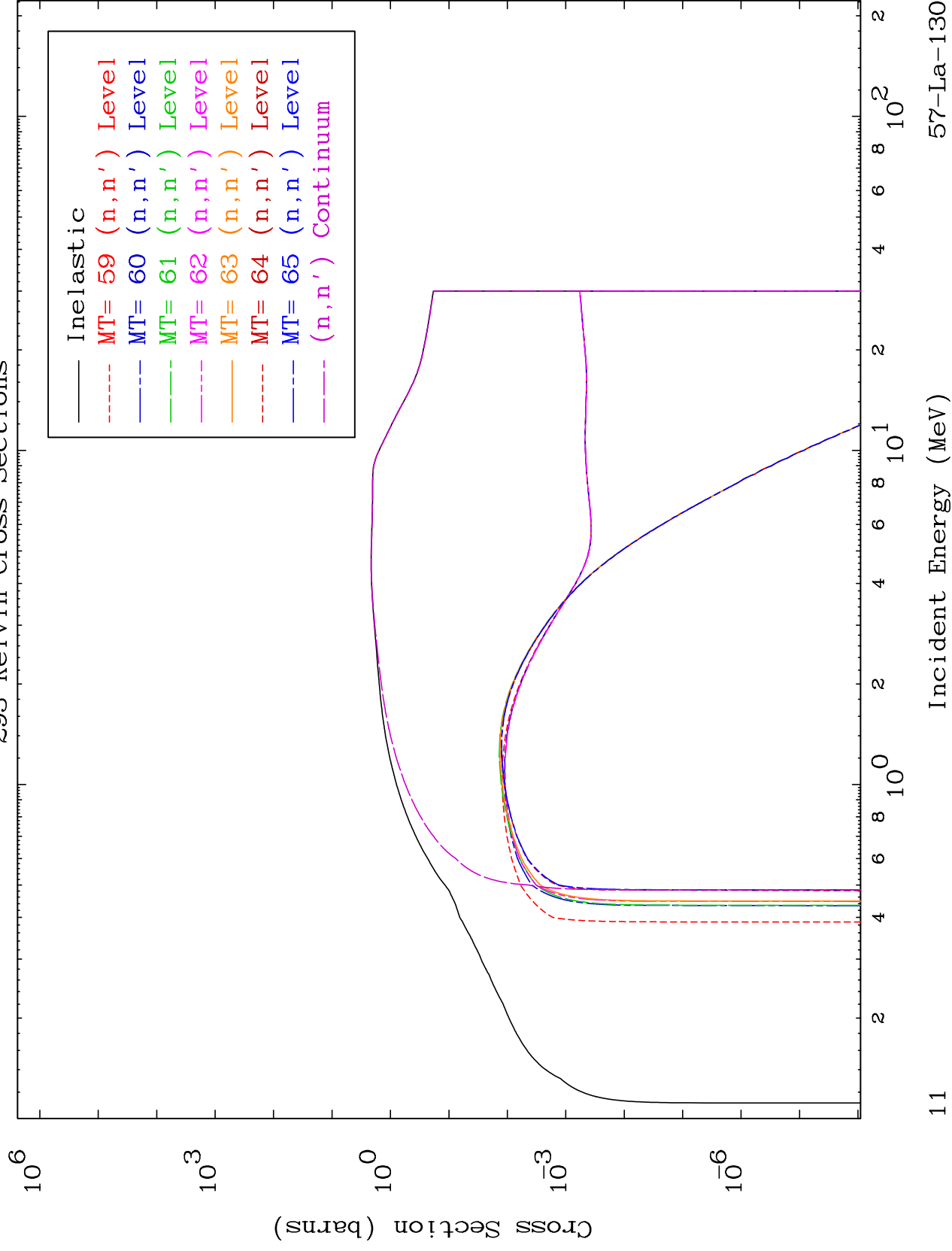
293 Kelvin Cross Sections



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Incident Energy (MeV)

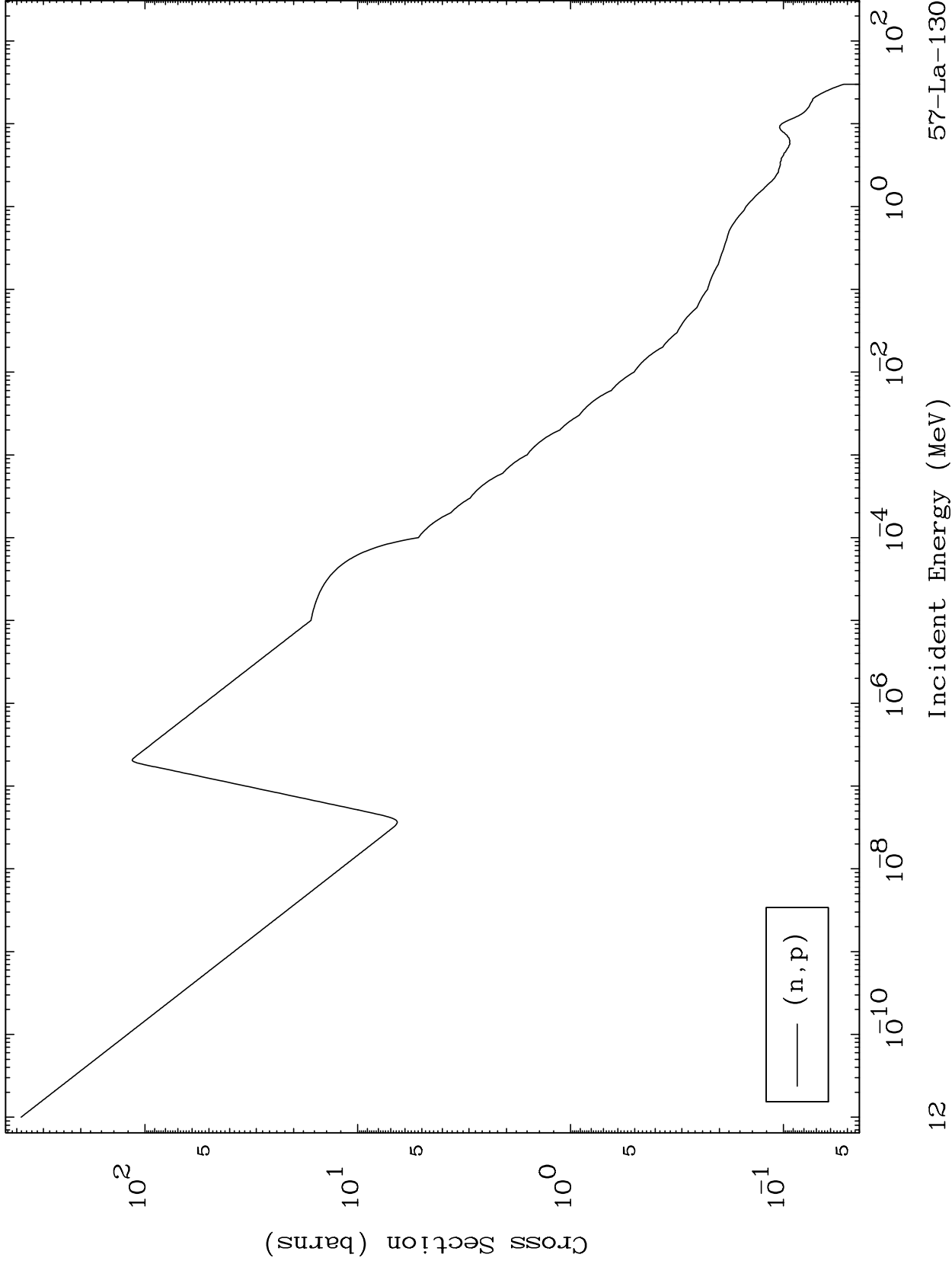
57-La-130



MAT 5701

(n,p) Levels  
293 Kelvin Cross Sections

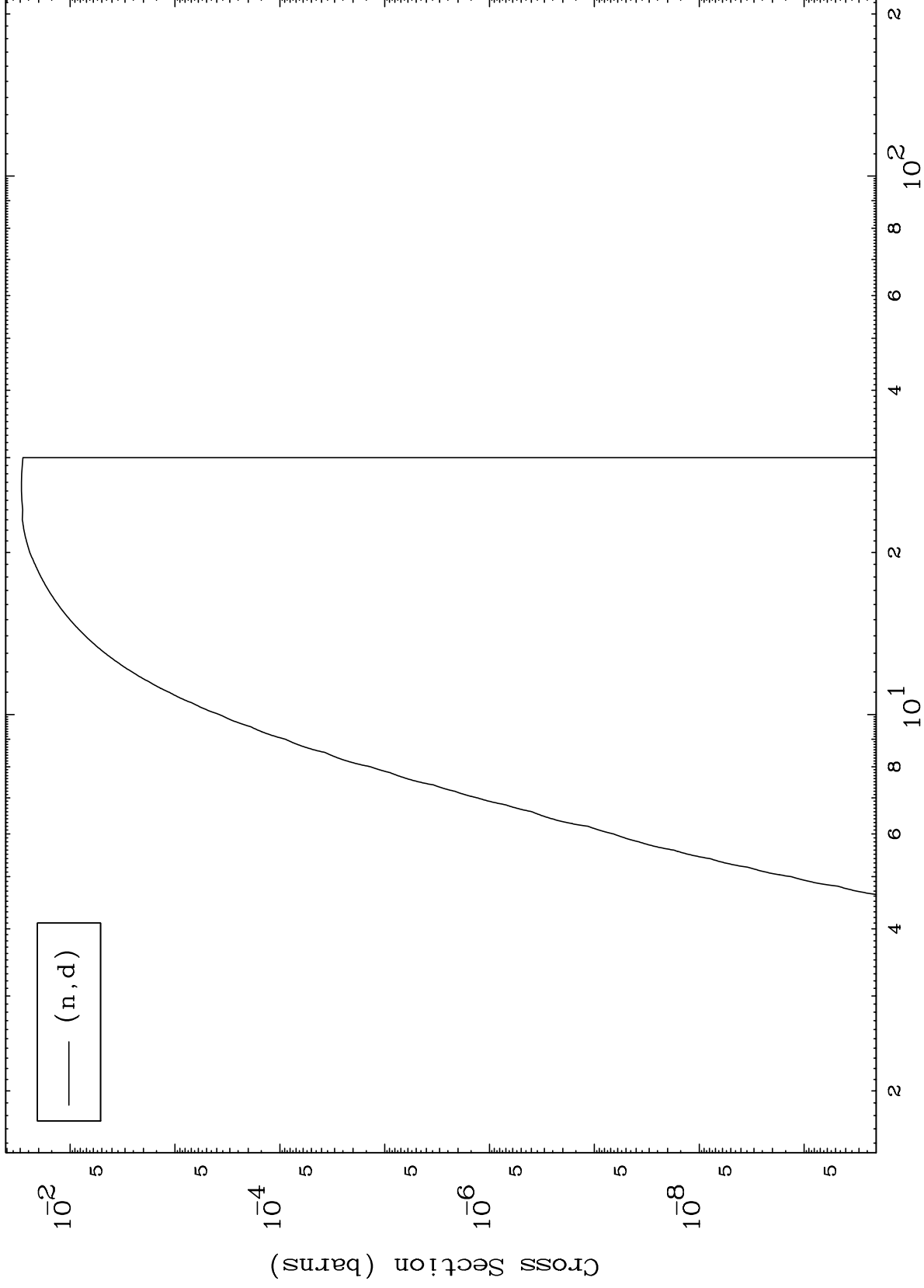
57-La-130



MAT 5701

(n,d) Levels  
293 Kelvin Cross Sections

57-La-130



13

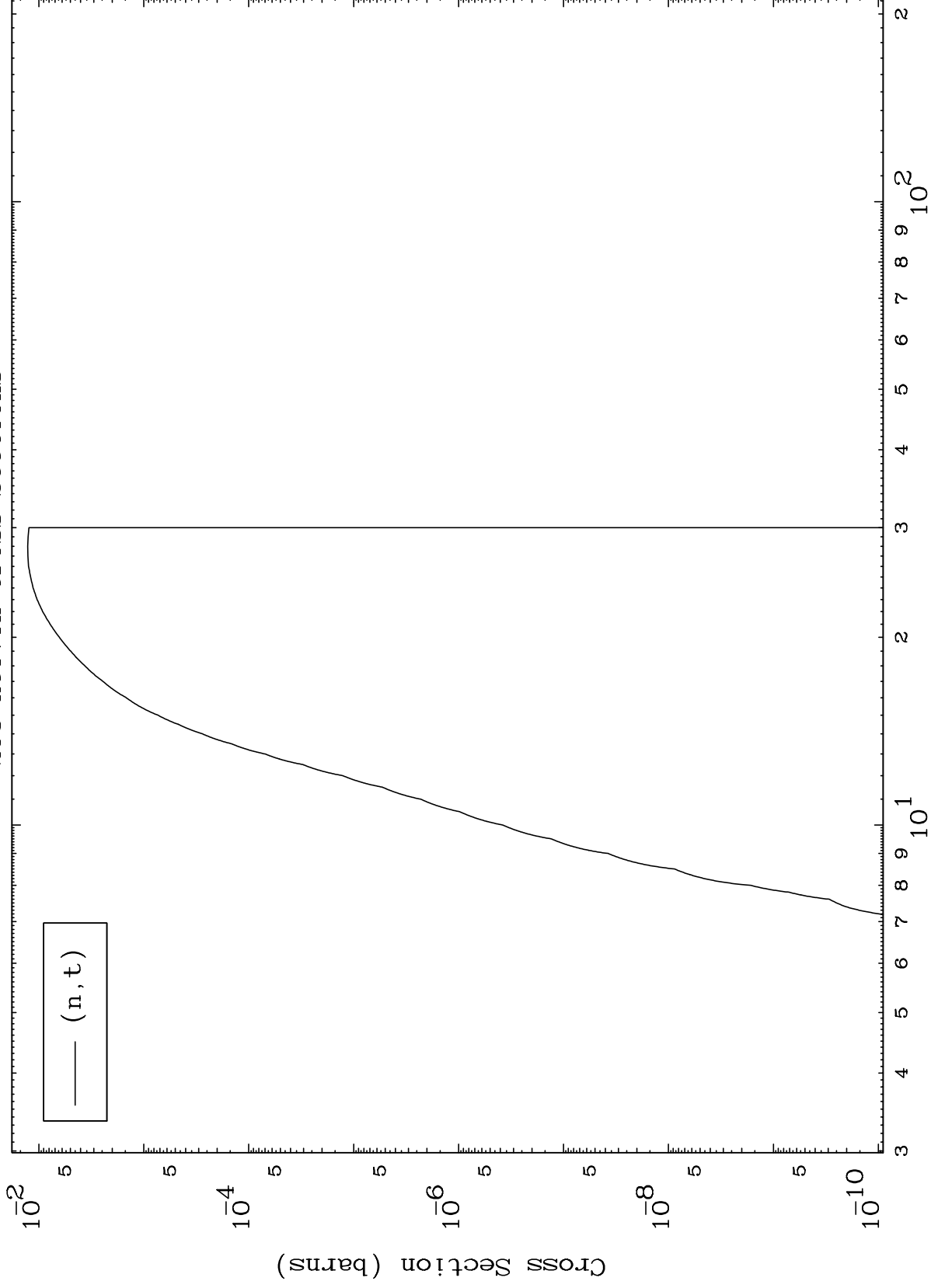
Incident Energy (MeV)

57-La-130

MAT 5701

(n,t) Levels  
293 Kelvin Cross Sections

57-La-130



14

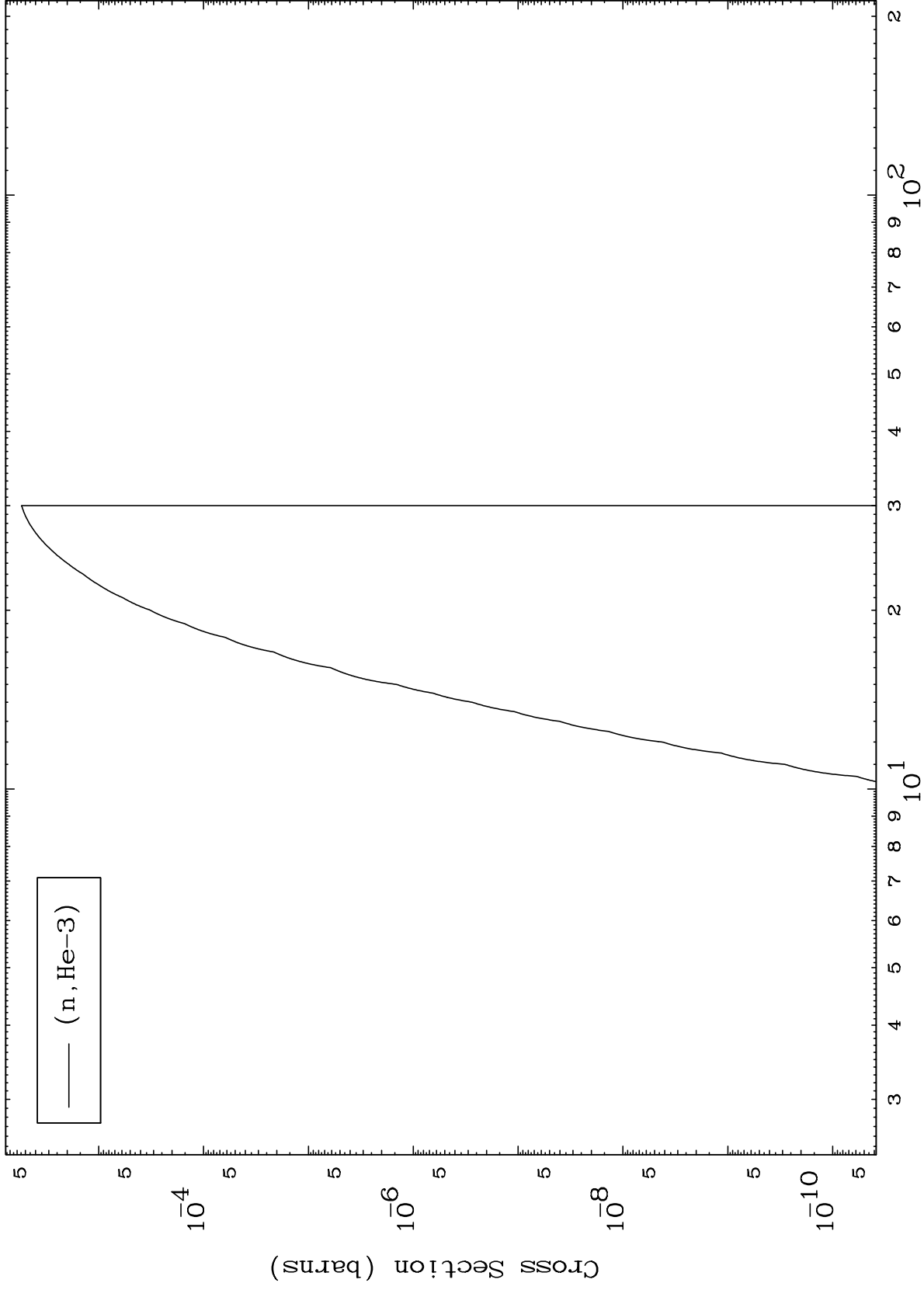
Incident Energy (MeV)

57-La-130

MAT 5701

(n,He3) Levels  
293 Kelvin Cross Sections

57-La-130



15

Incident Energy (MeV)

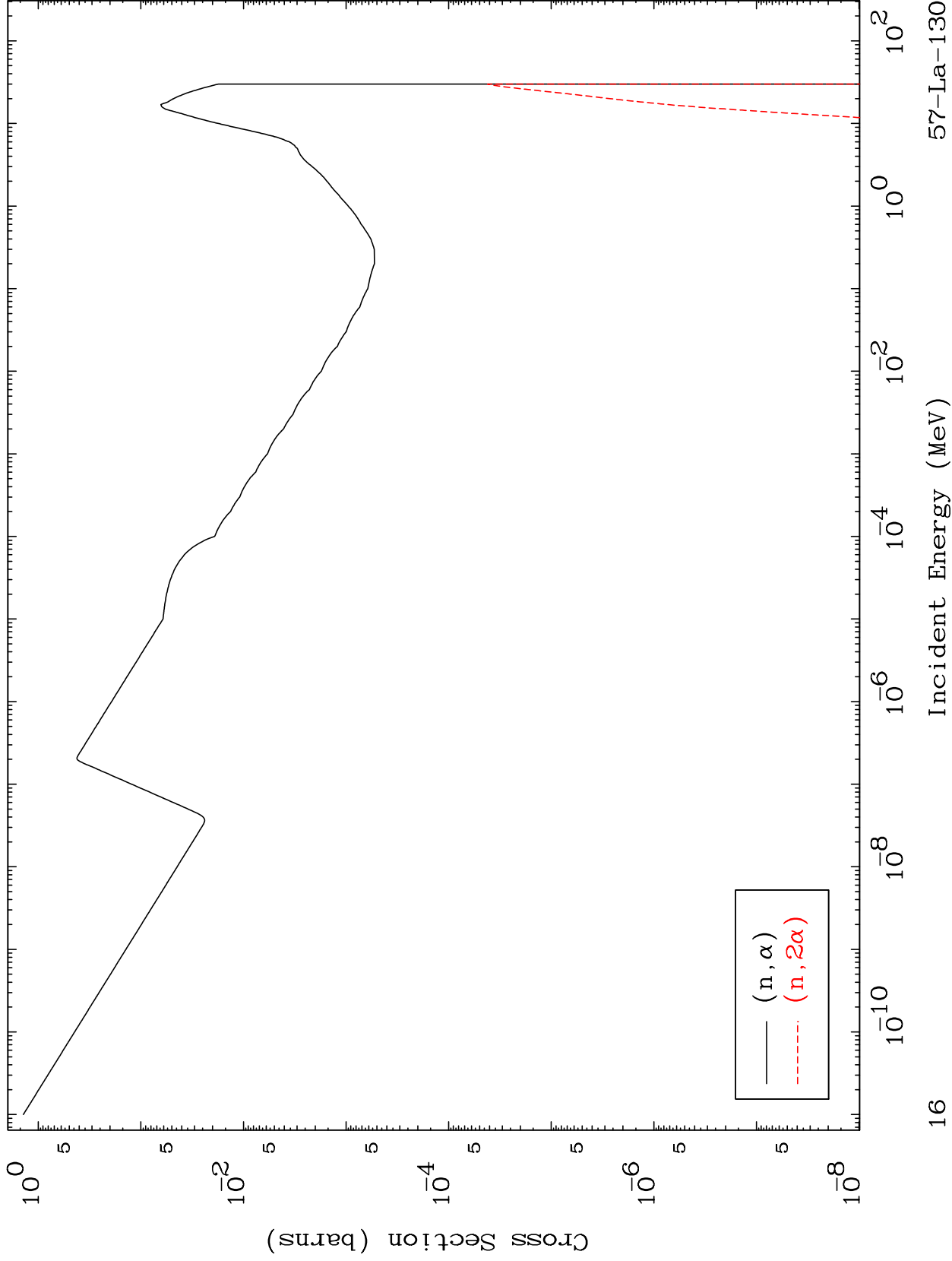
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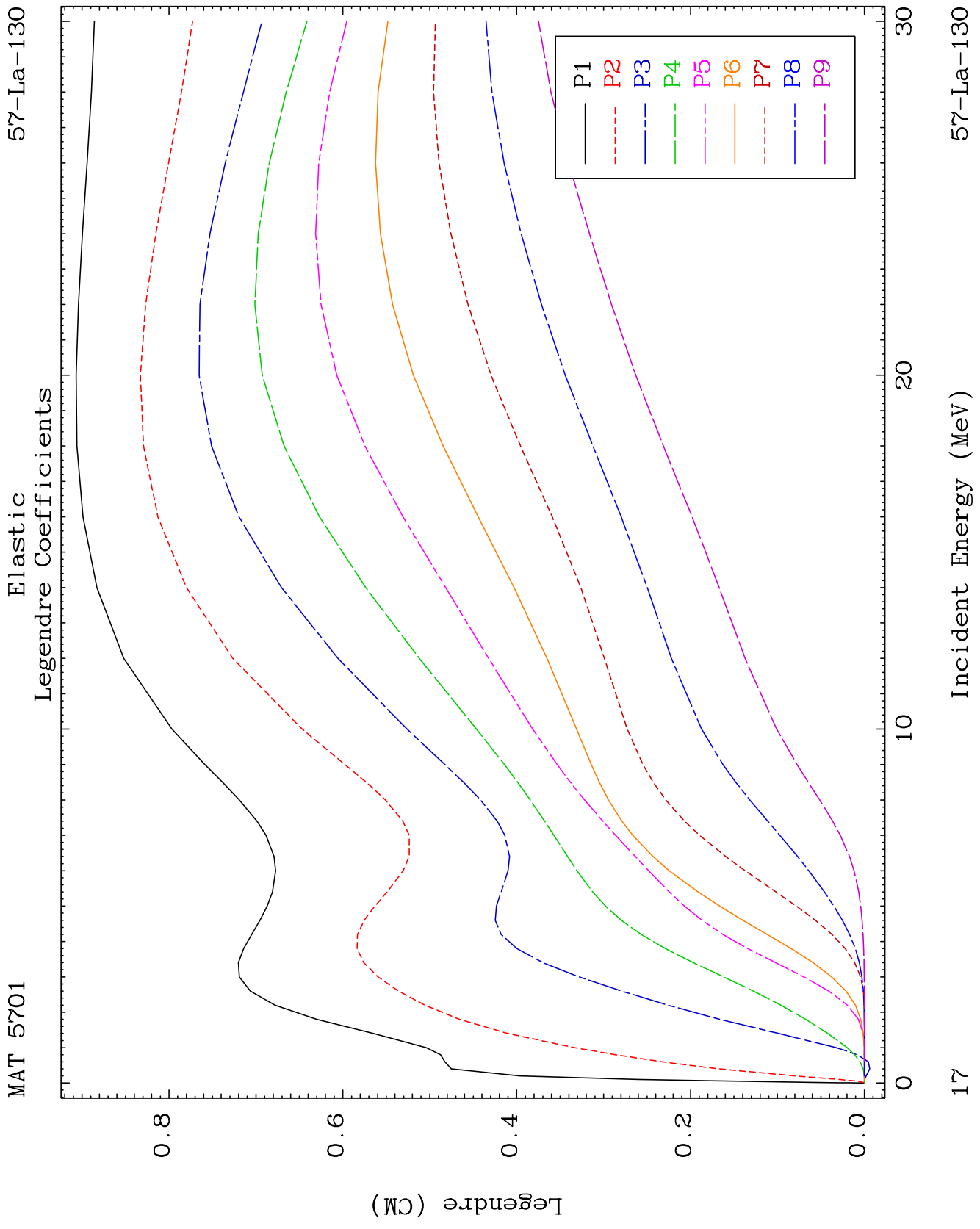


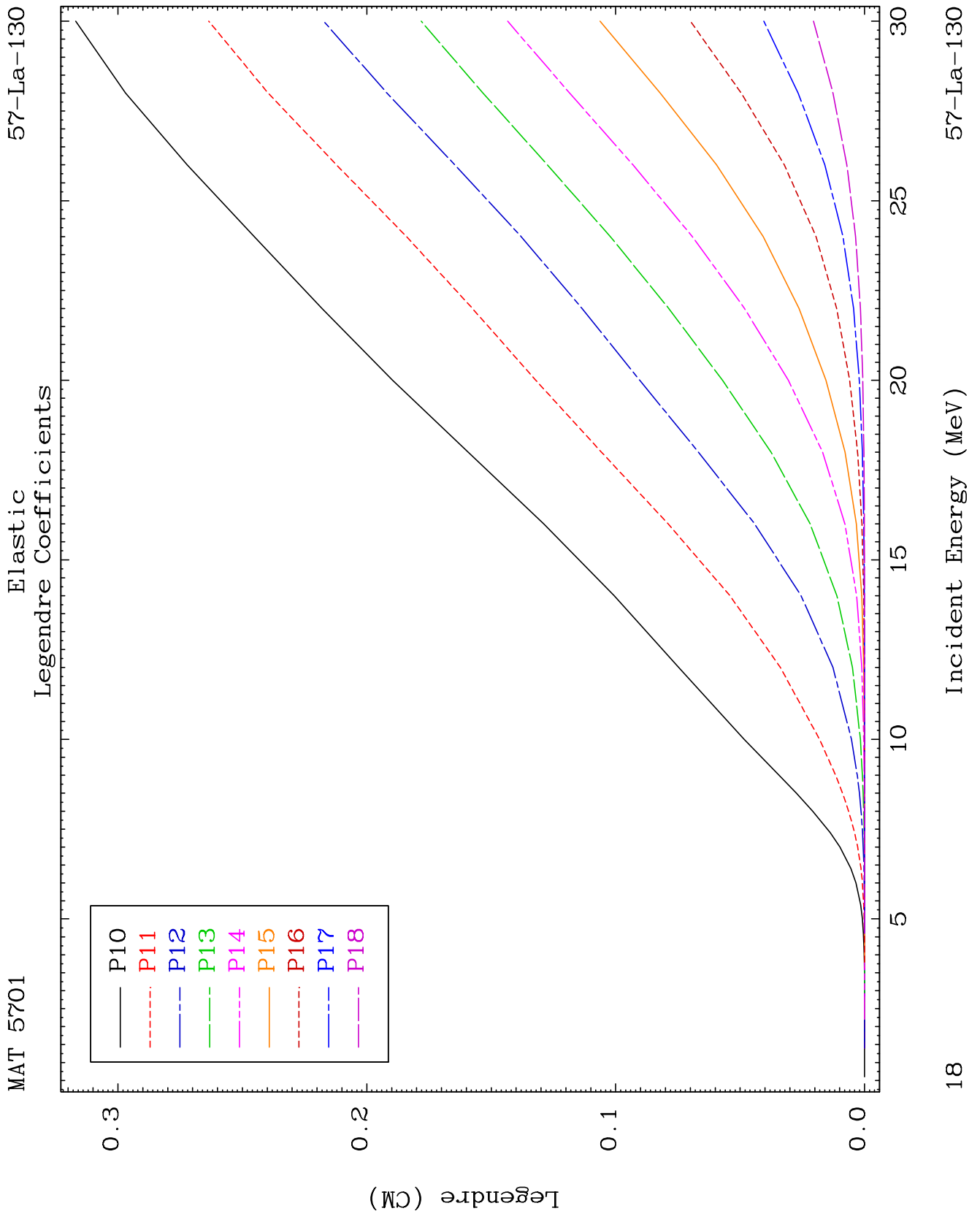
MAT 5701

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

57-La-130



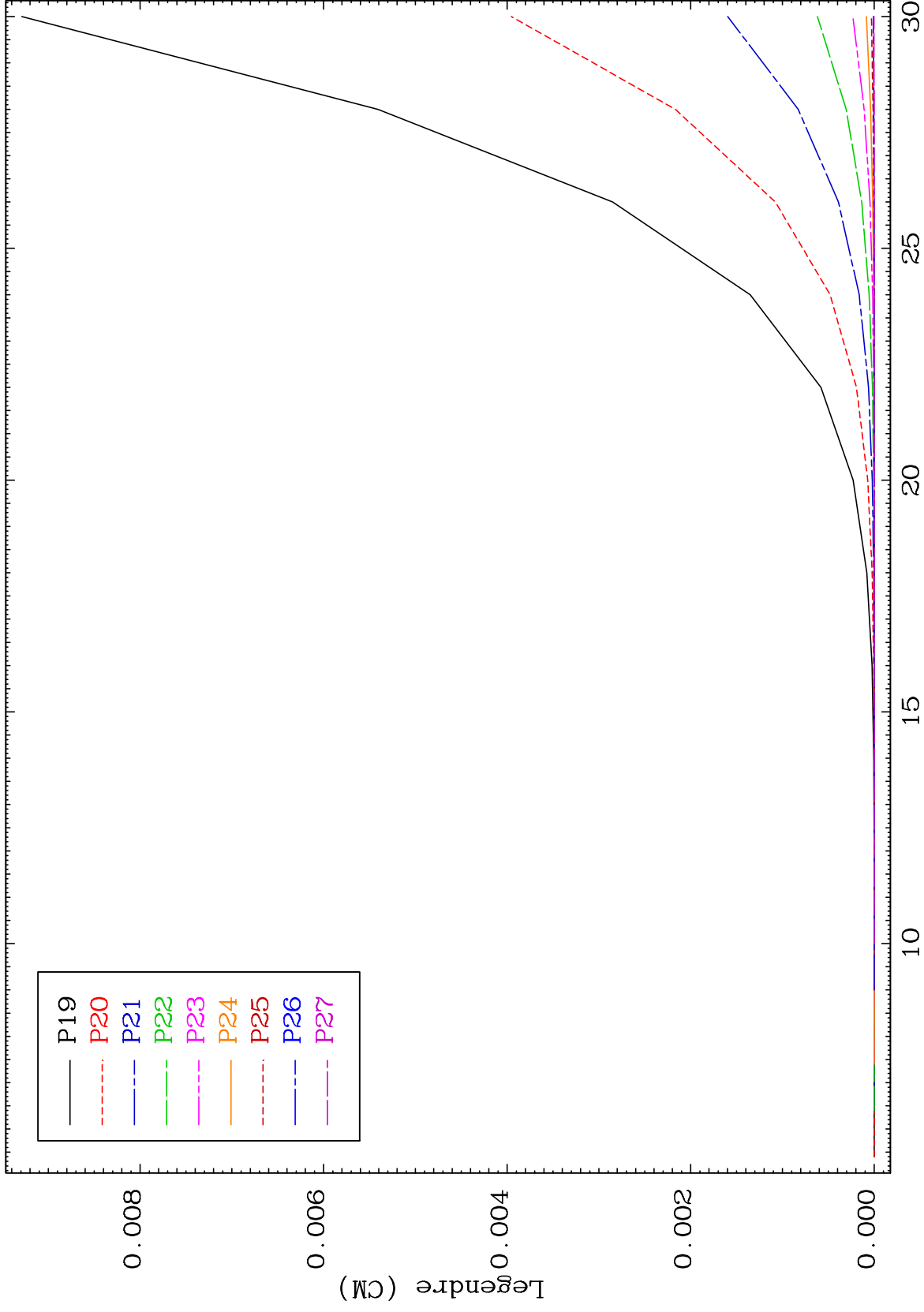




MAT 5701

Elastic Legendre Coefficients

57-La-130



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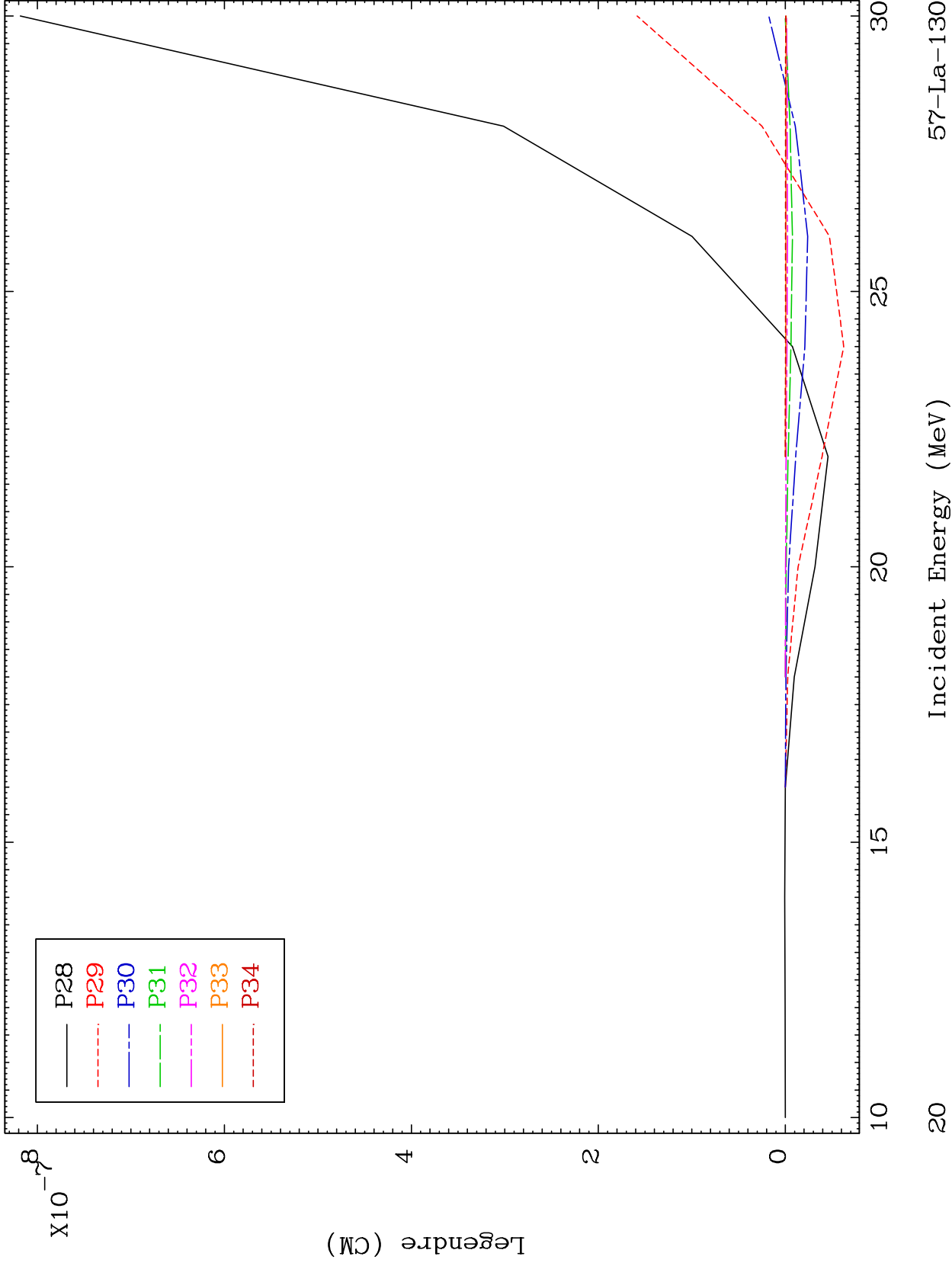
Incident Energy (MeV)

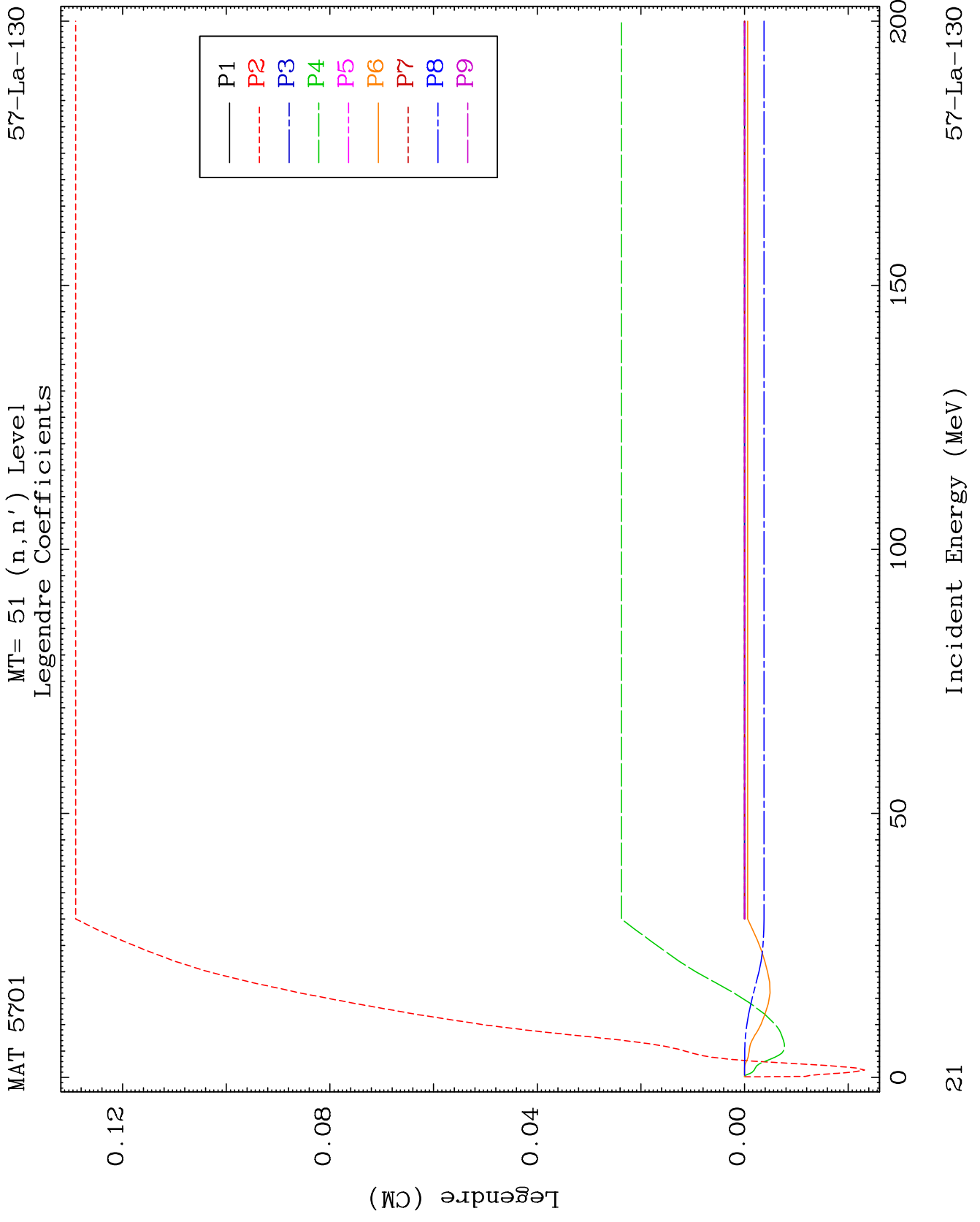
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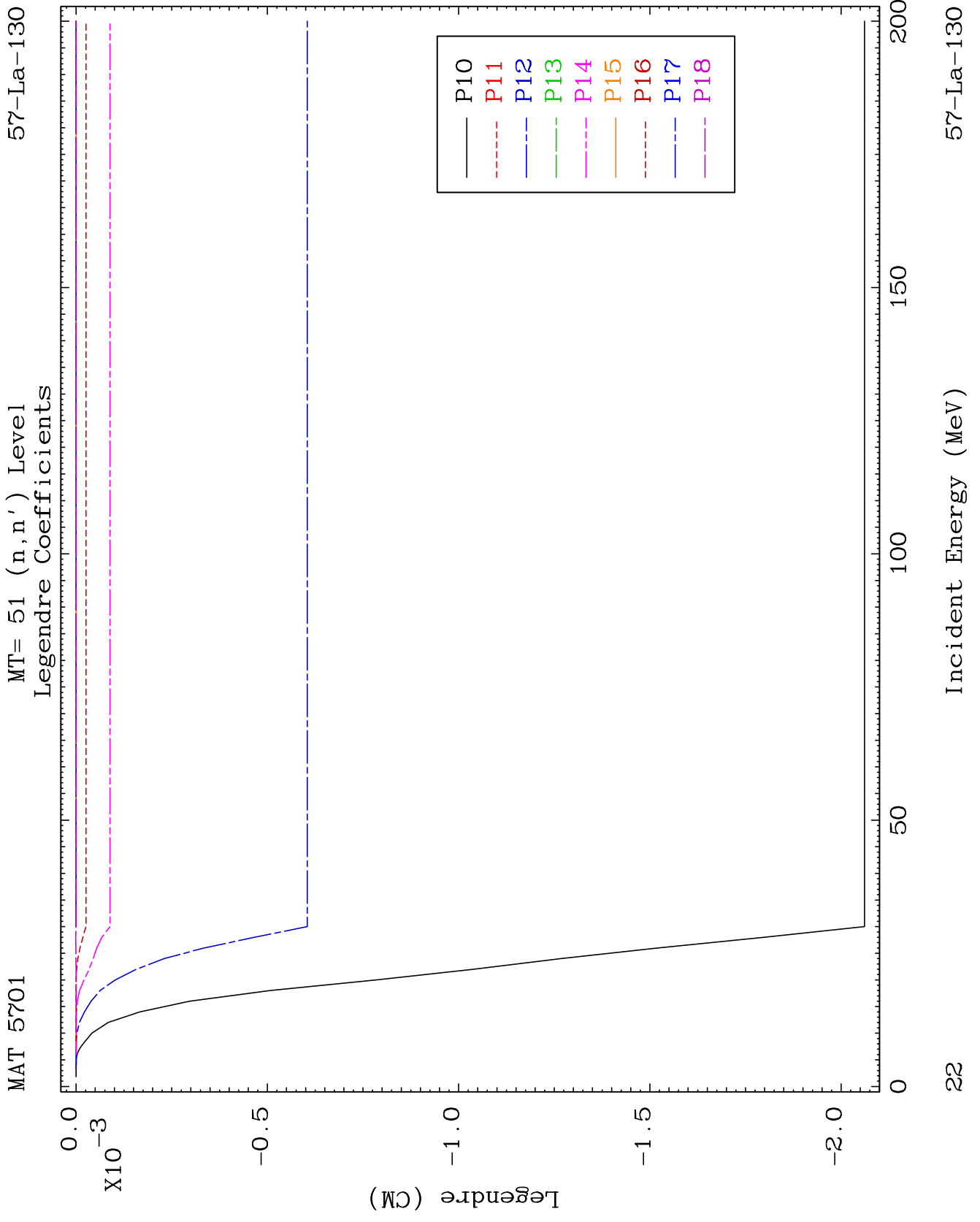
MAT 5701

Elastic  
Legendre Coefficients

57-La-130

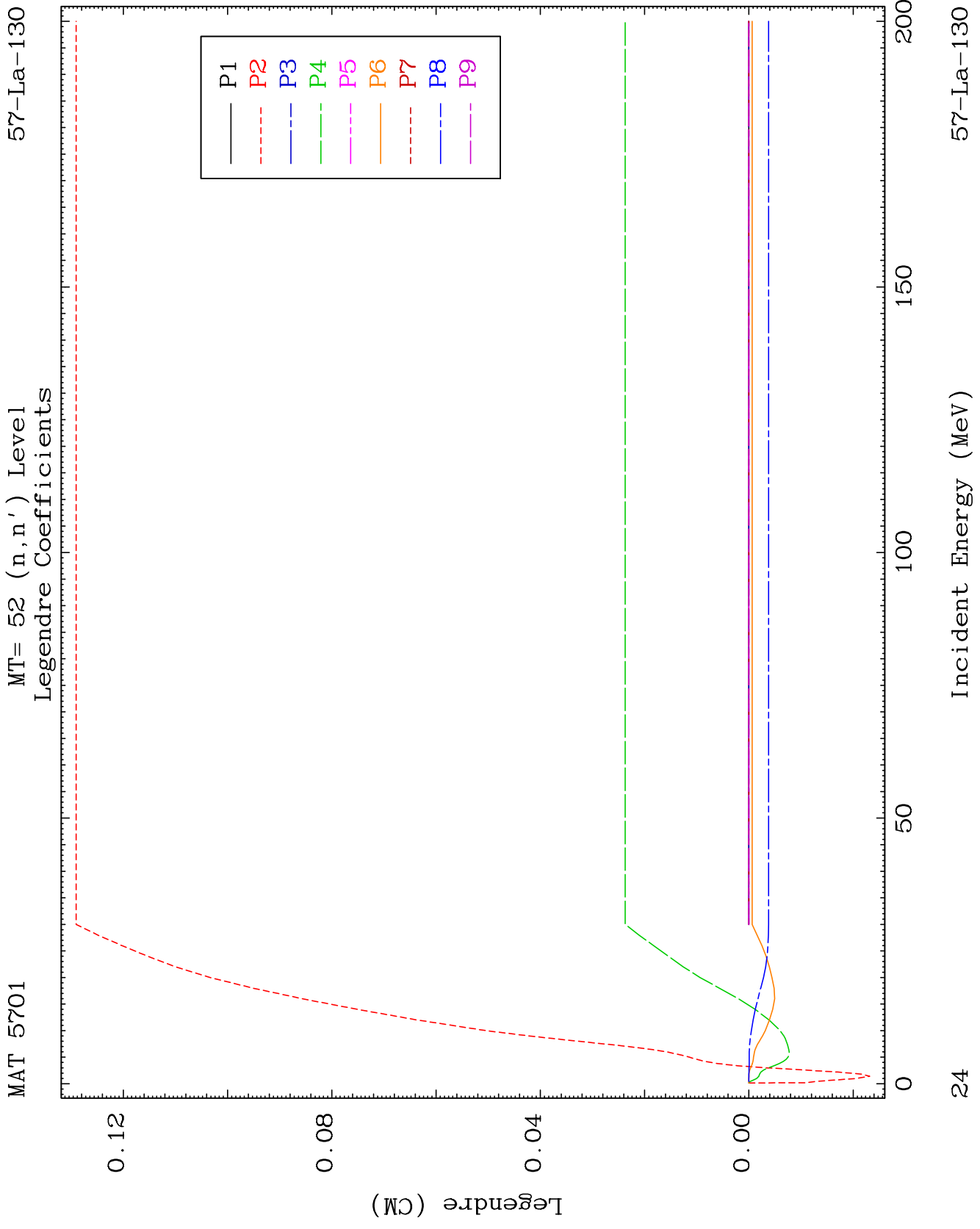


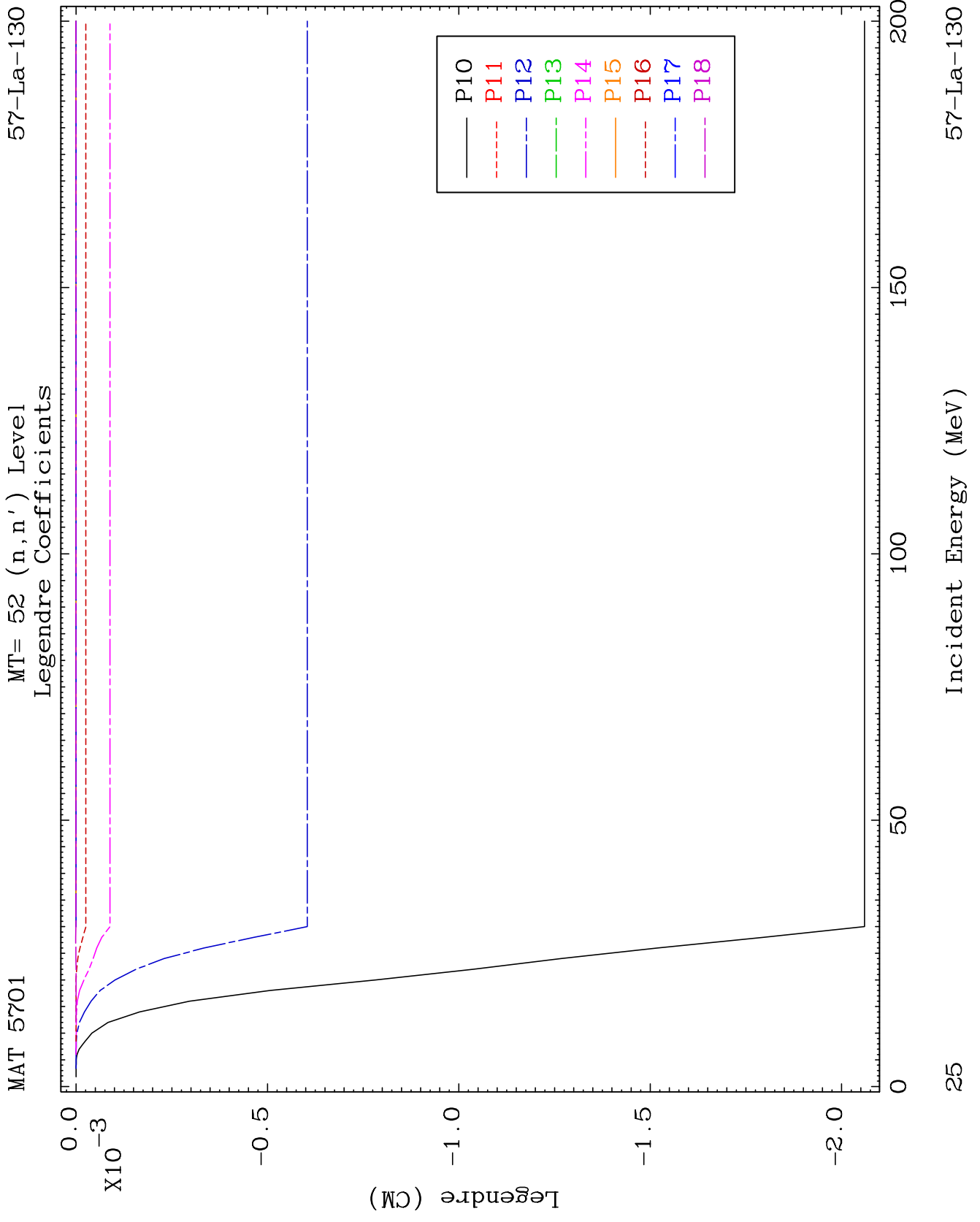


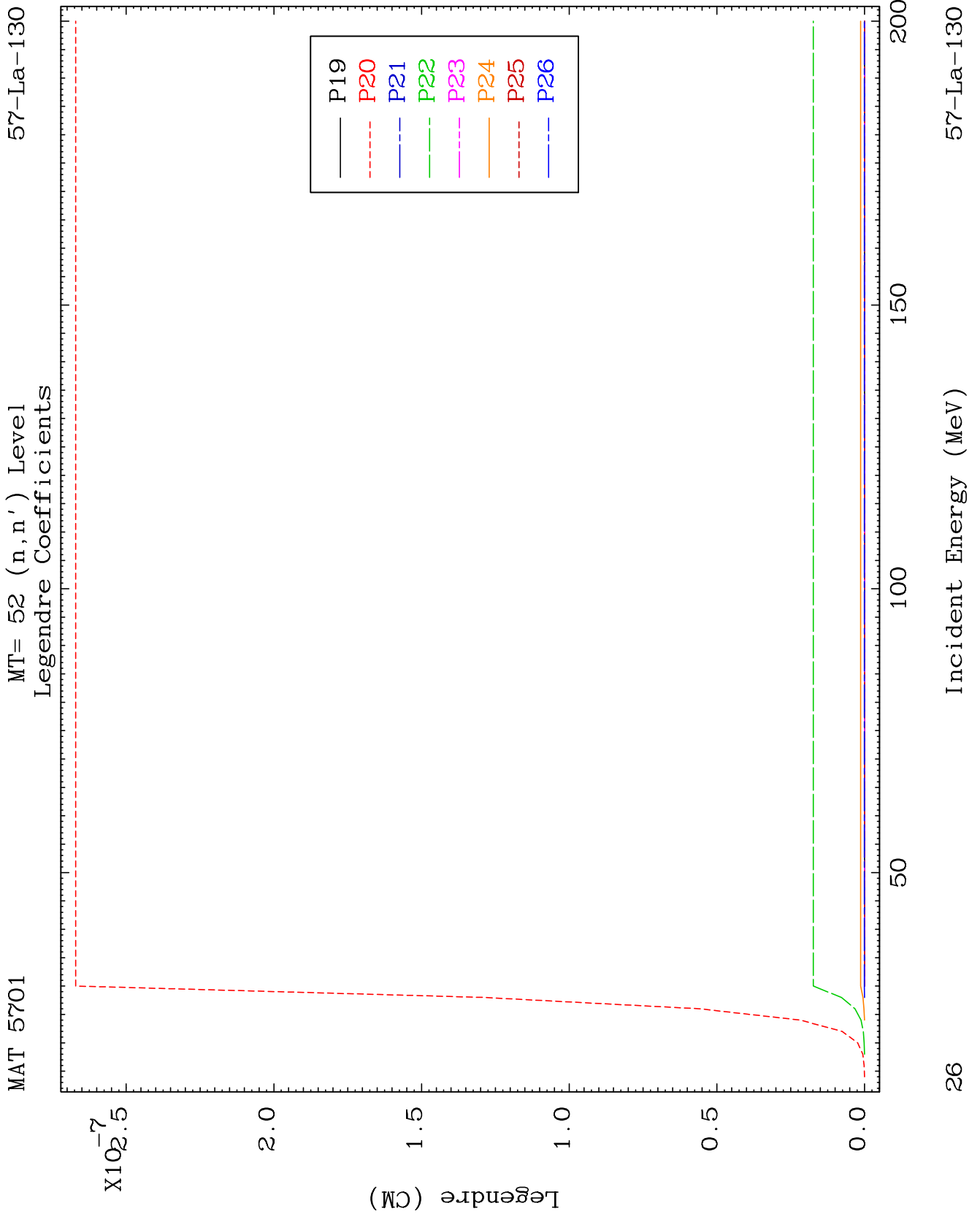




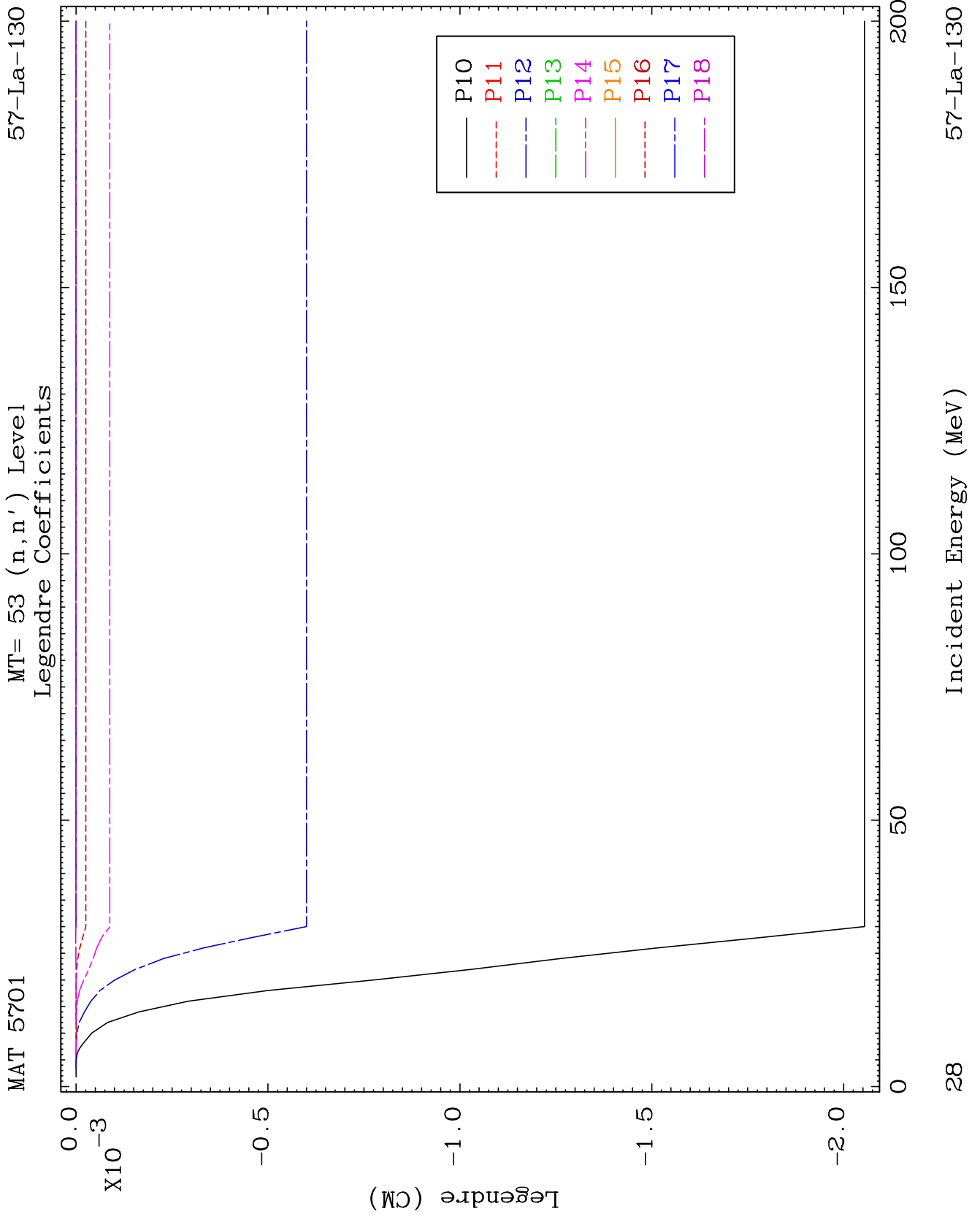


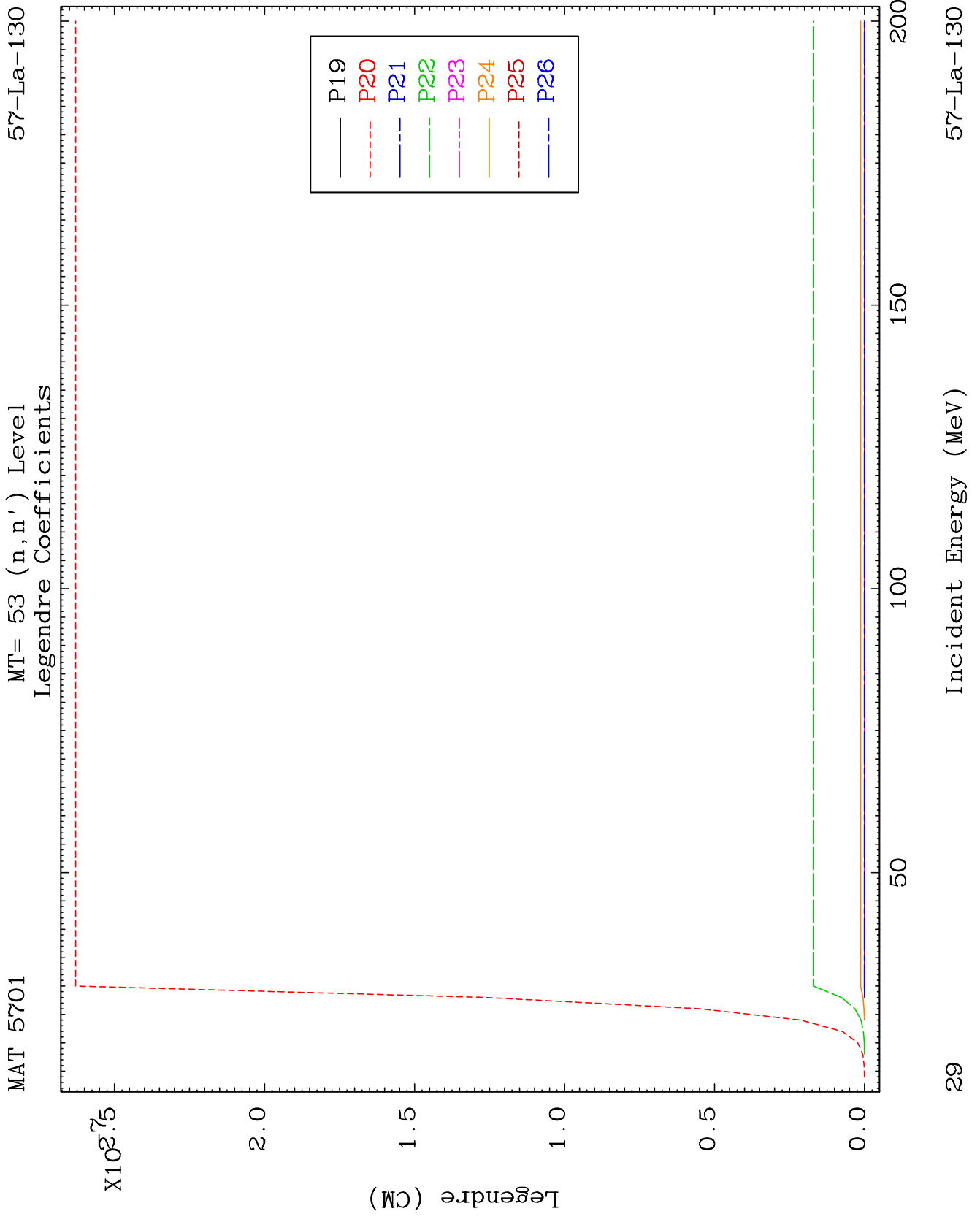


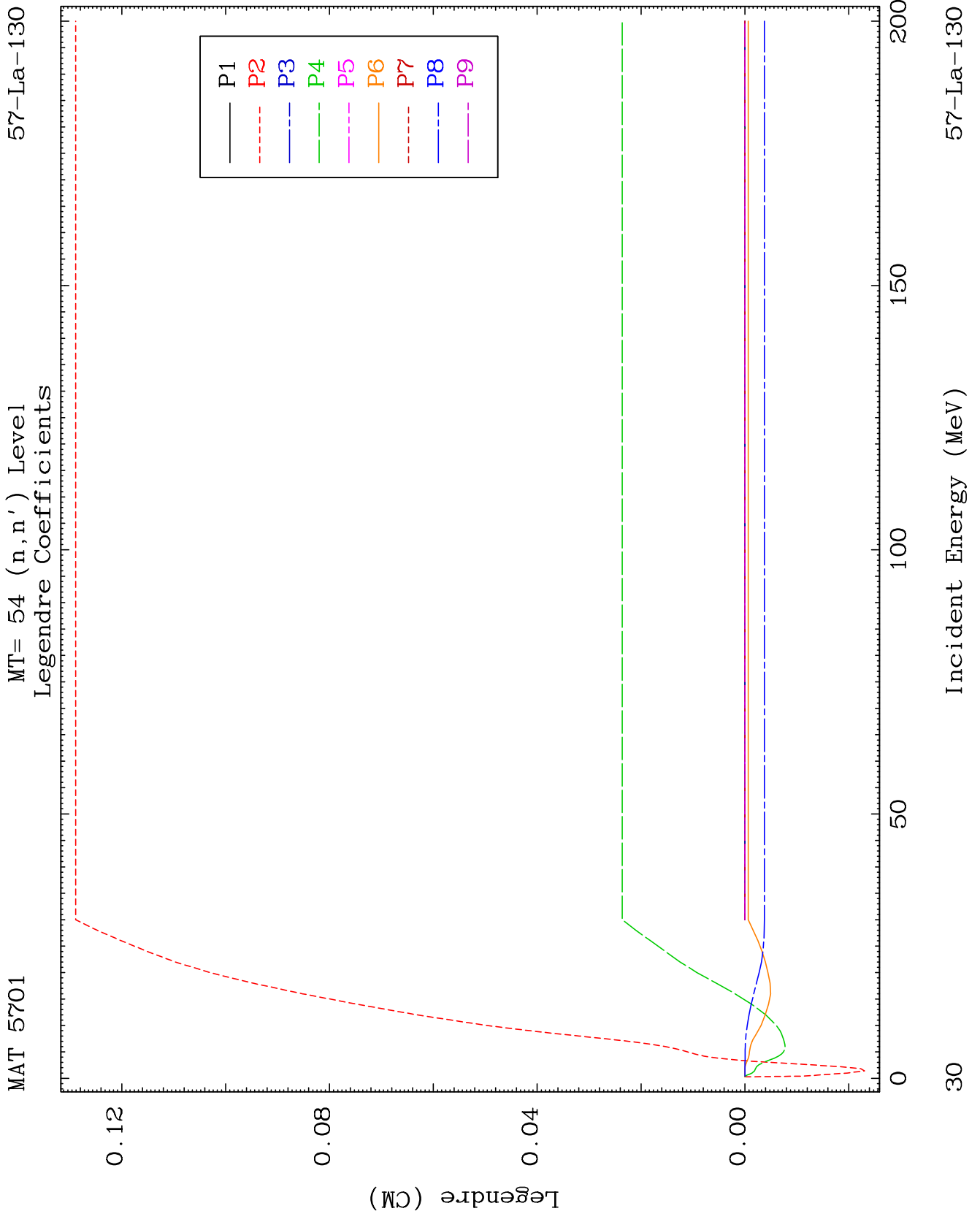


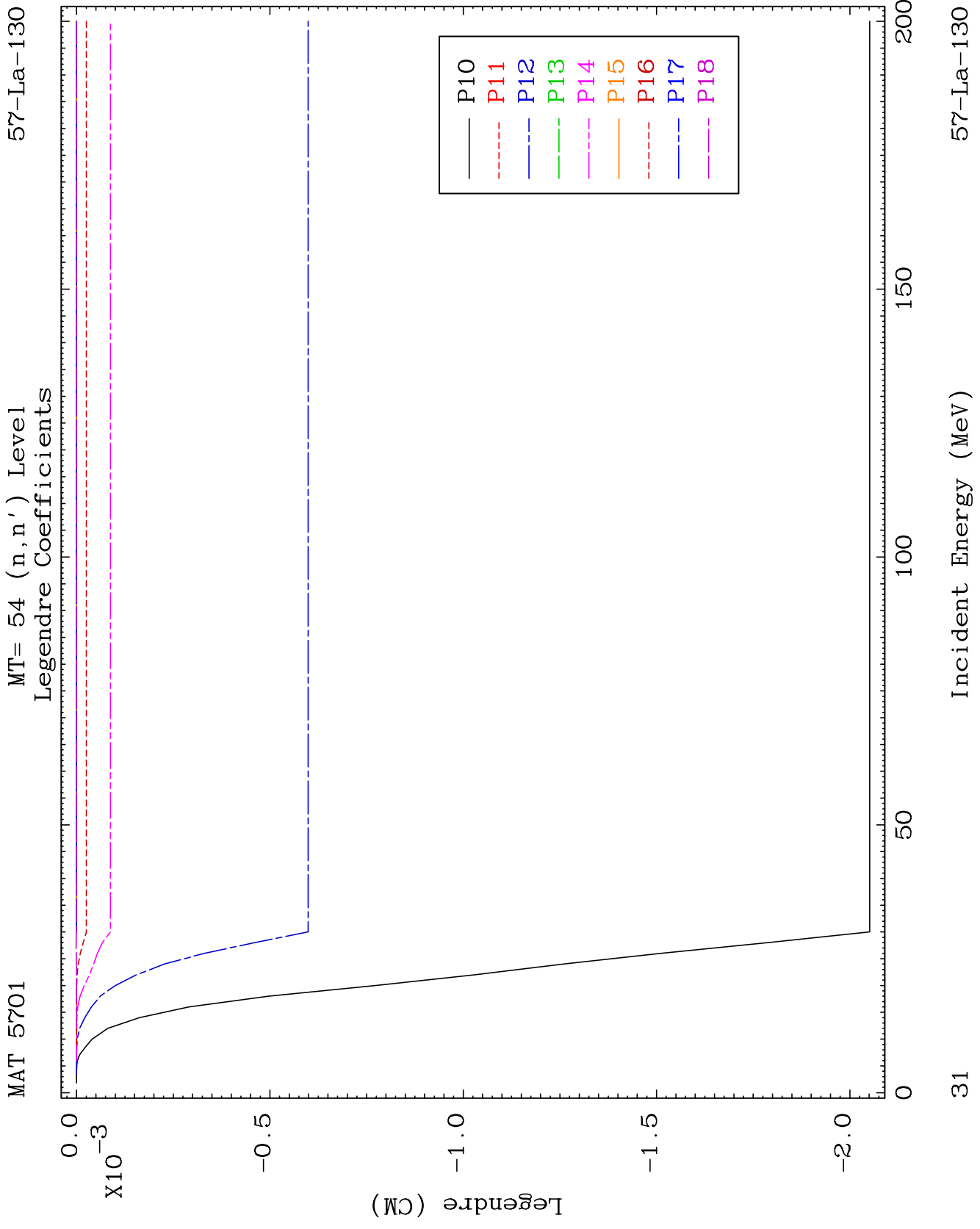




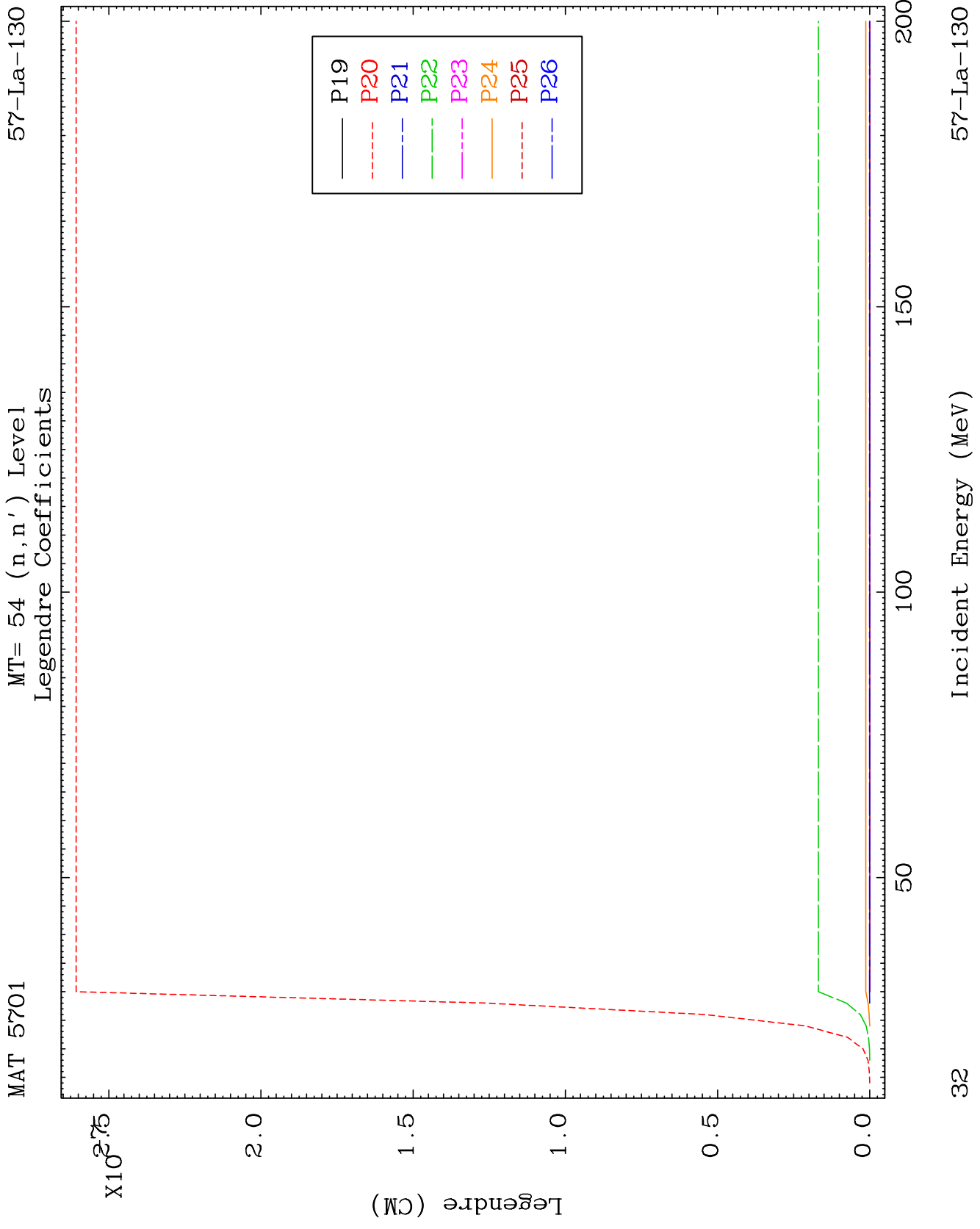








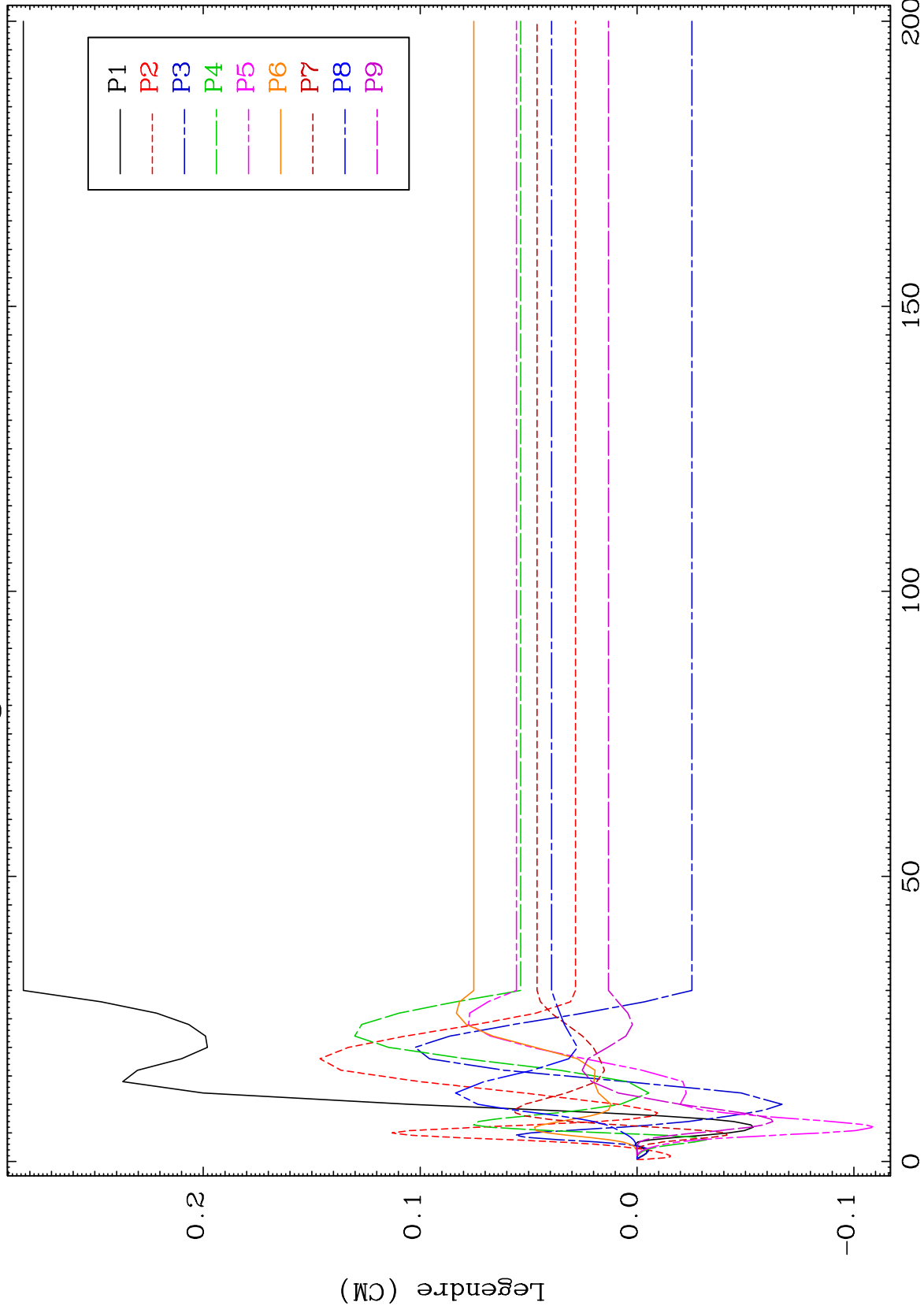




MAT 5701

MT= 55 (n,n') Level  
Legendre Coefficients

57-La-130

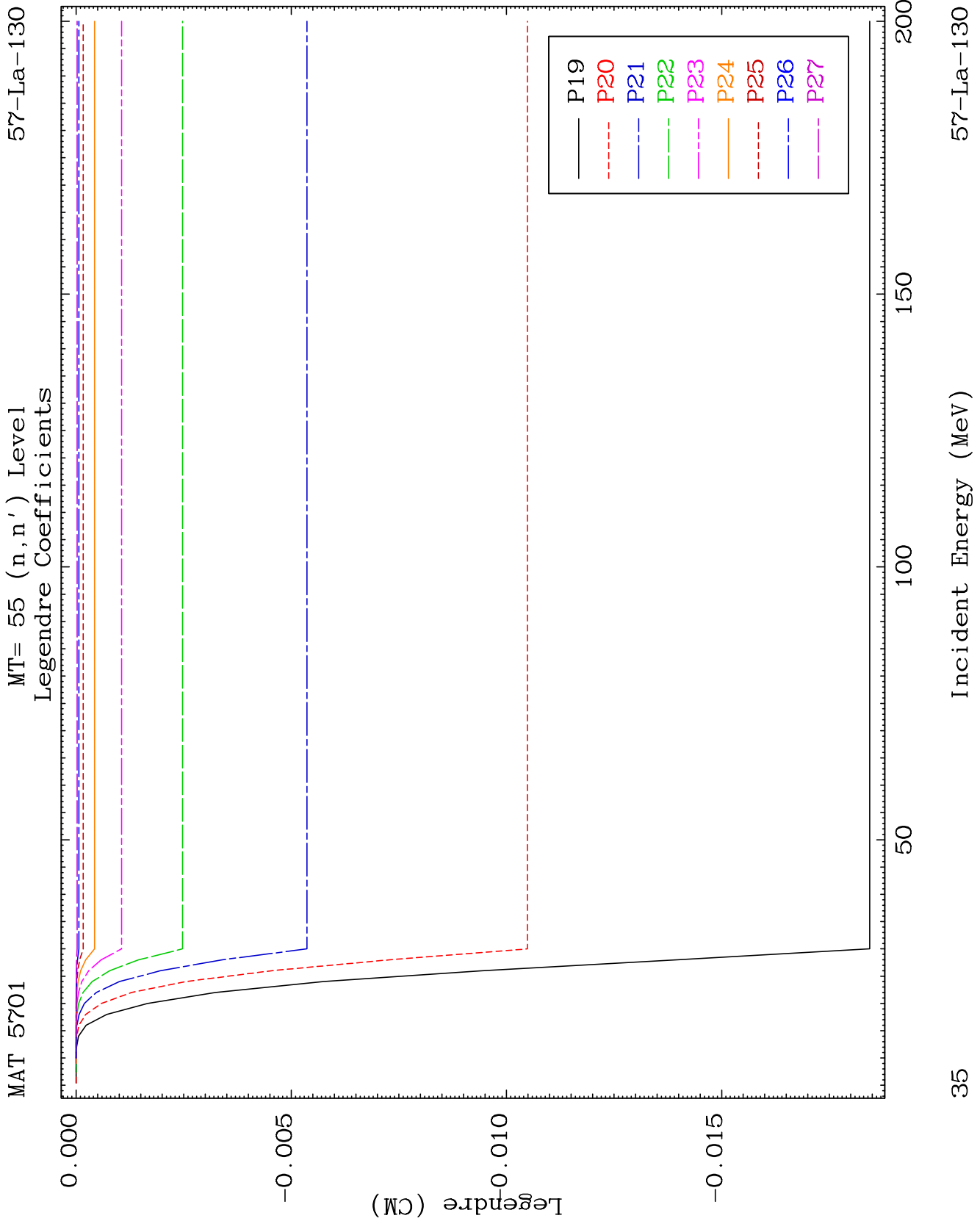


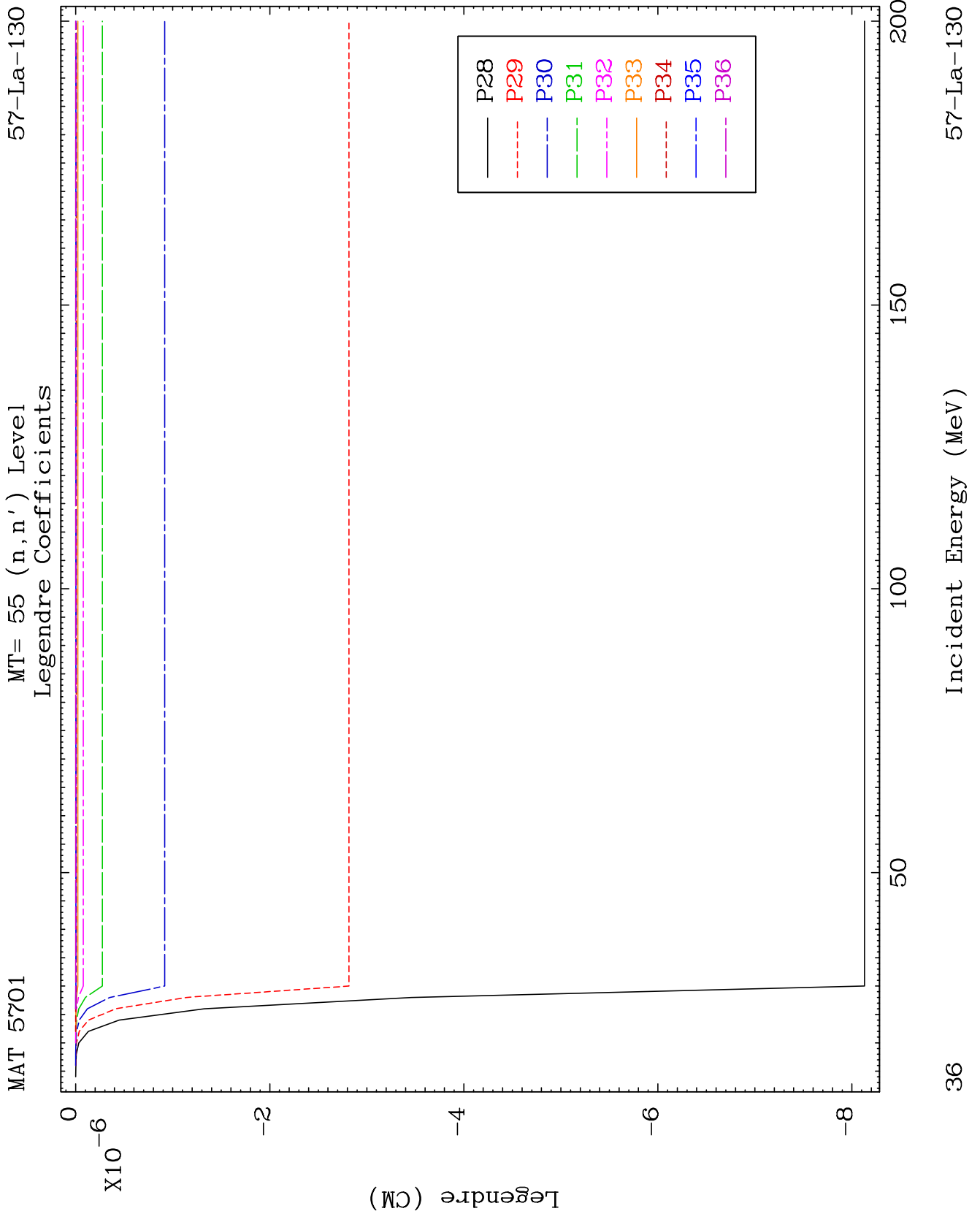
57-La-130

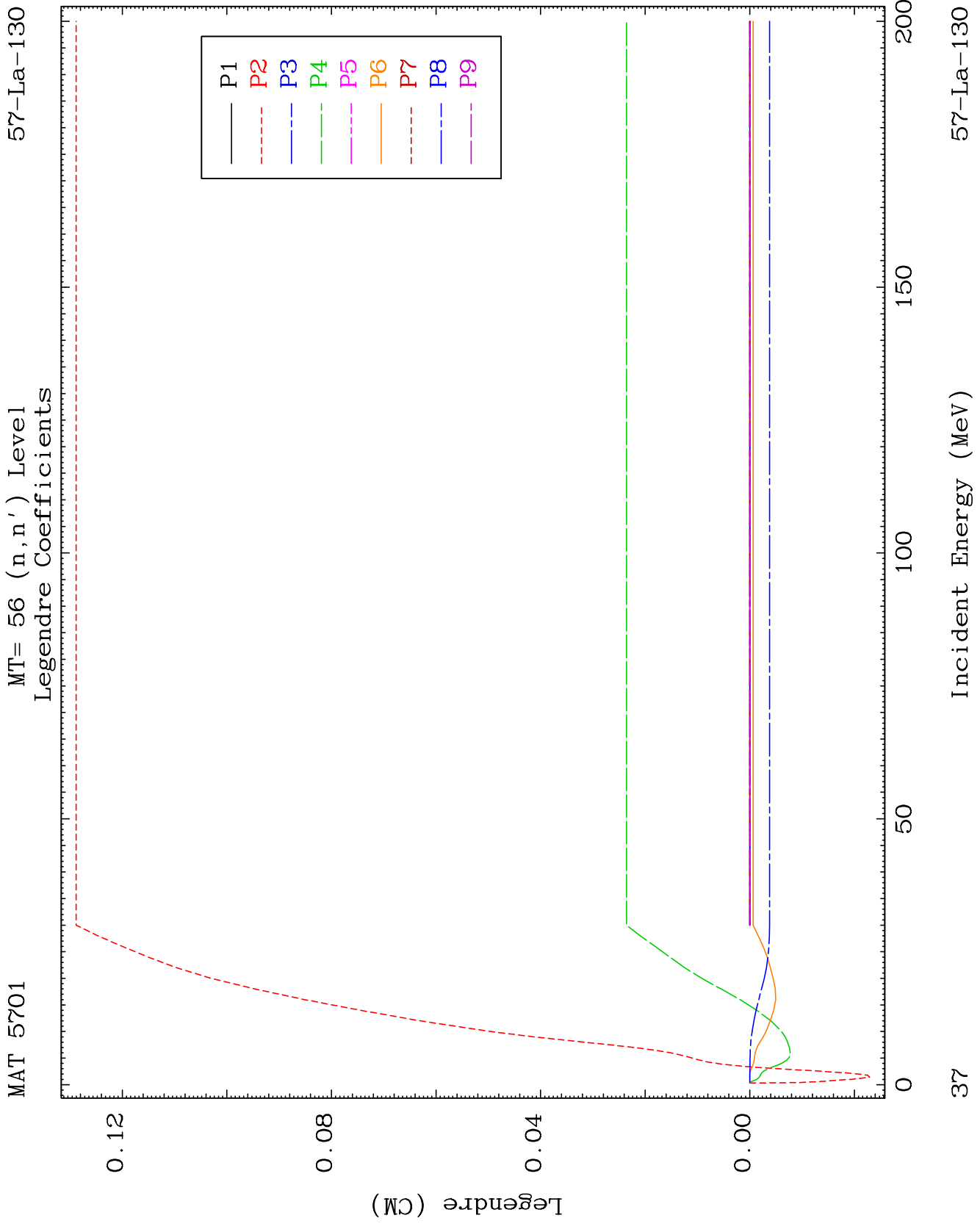
Incident Energy (MeV)

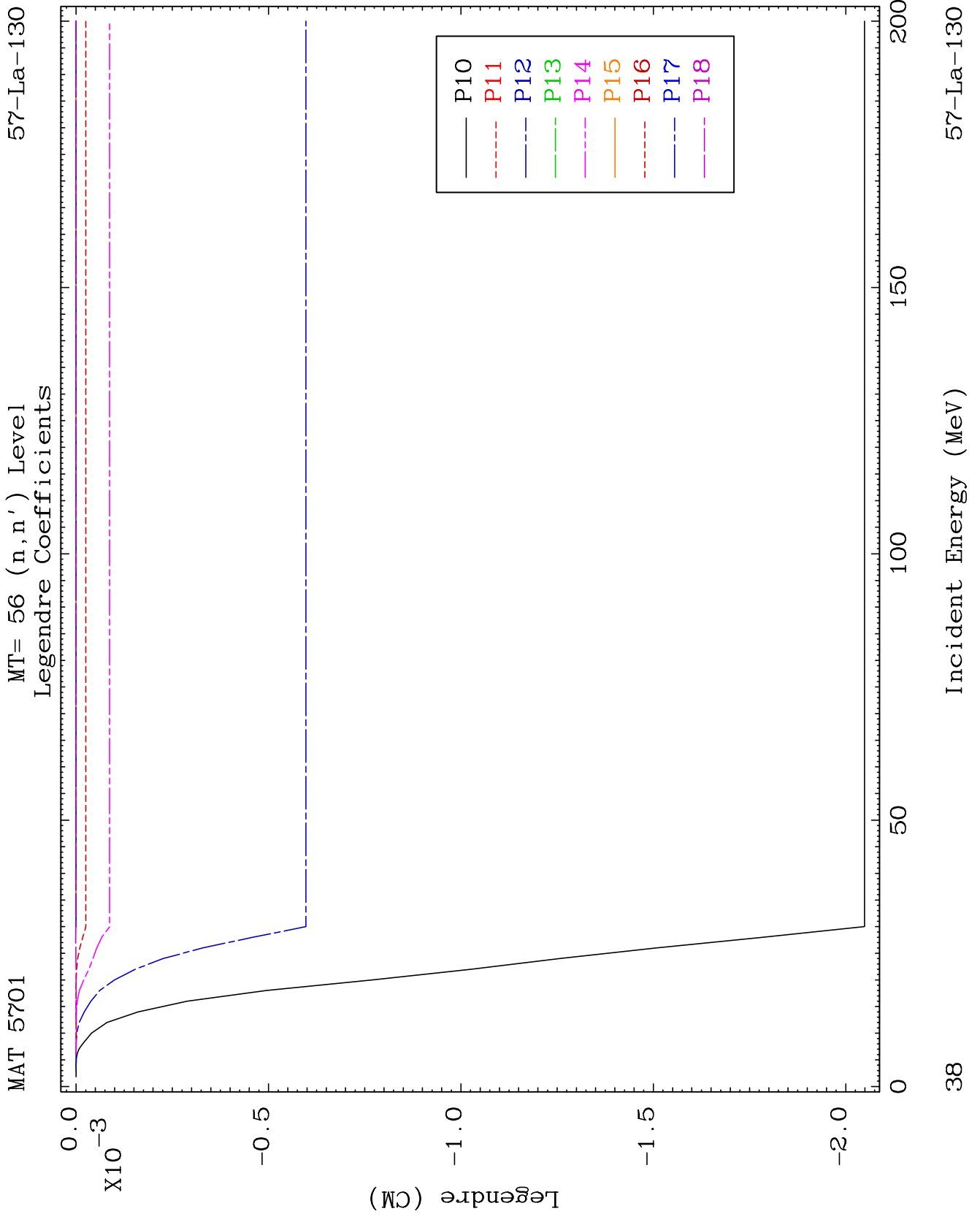
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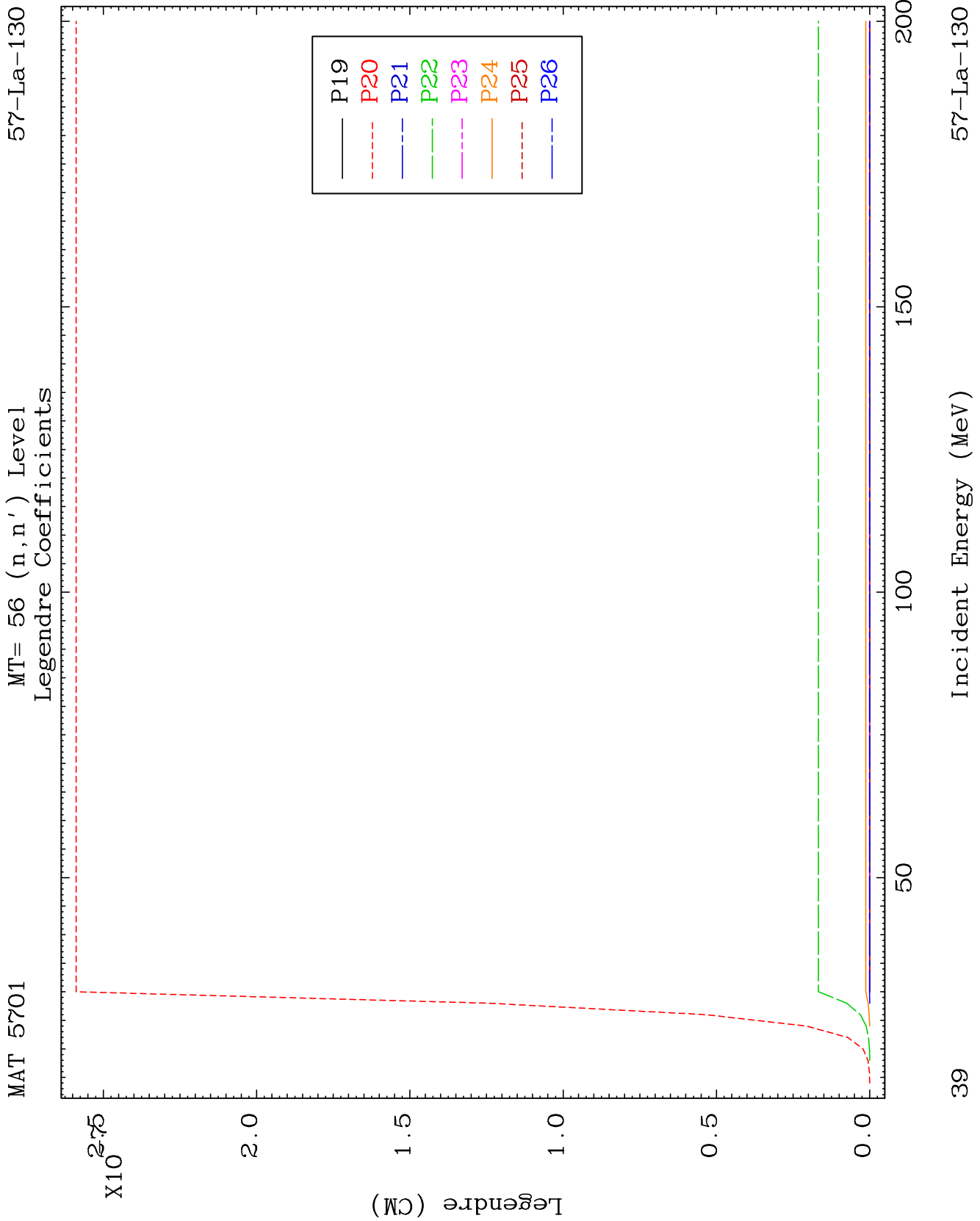




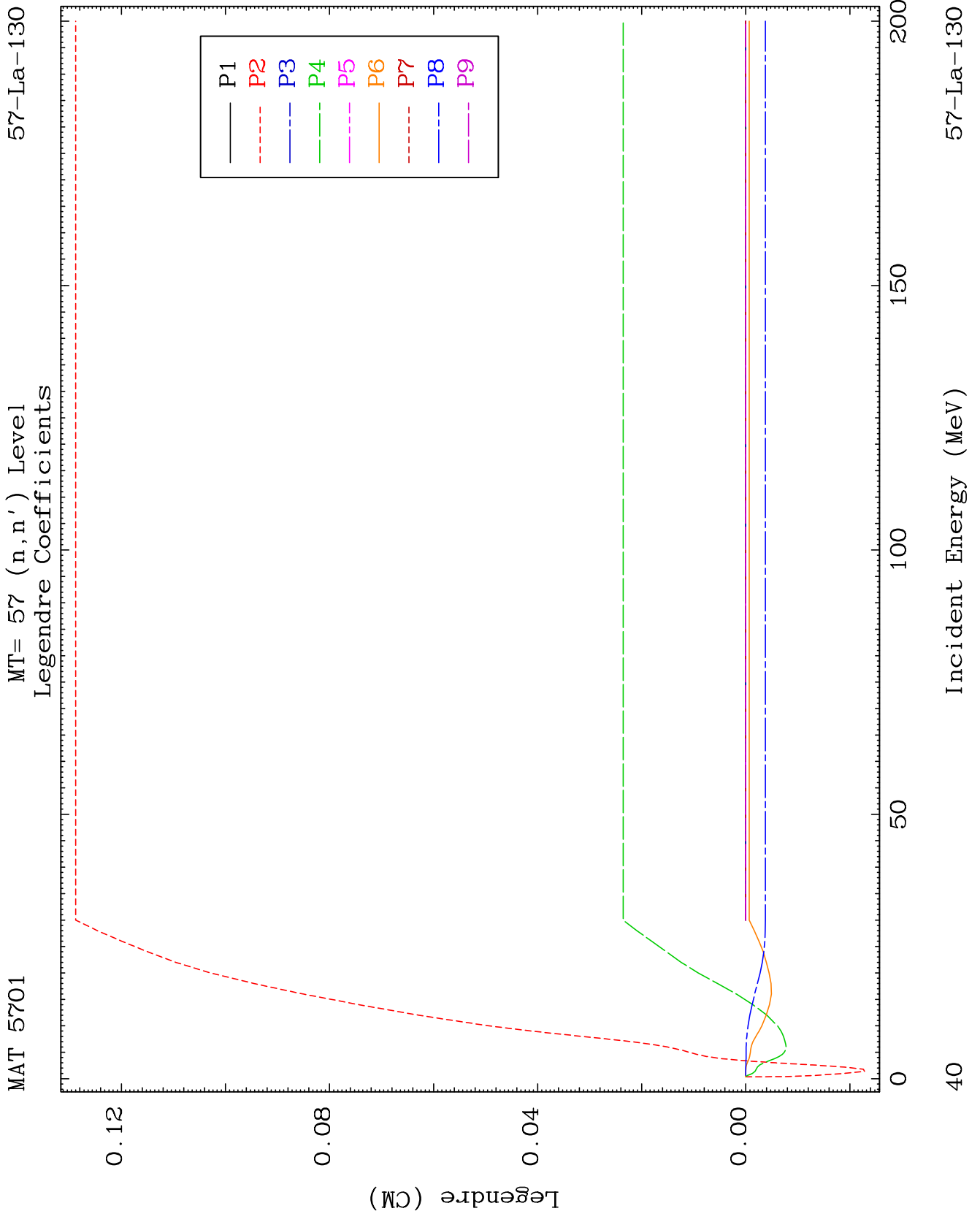


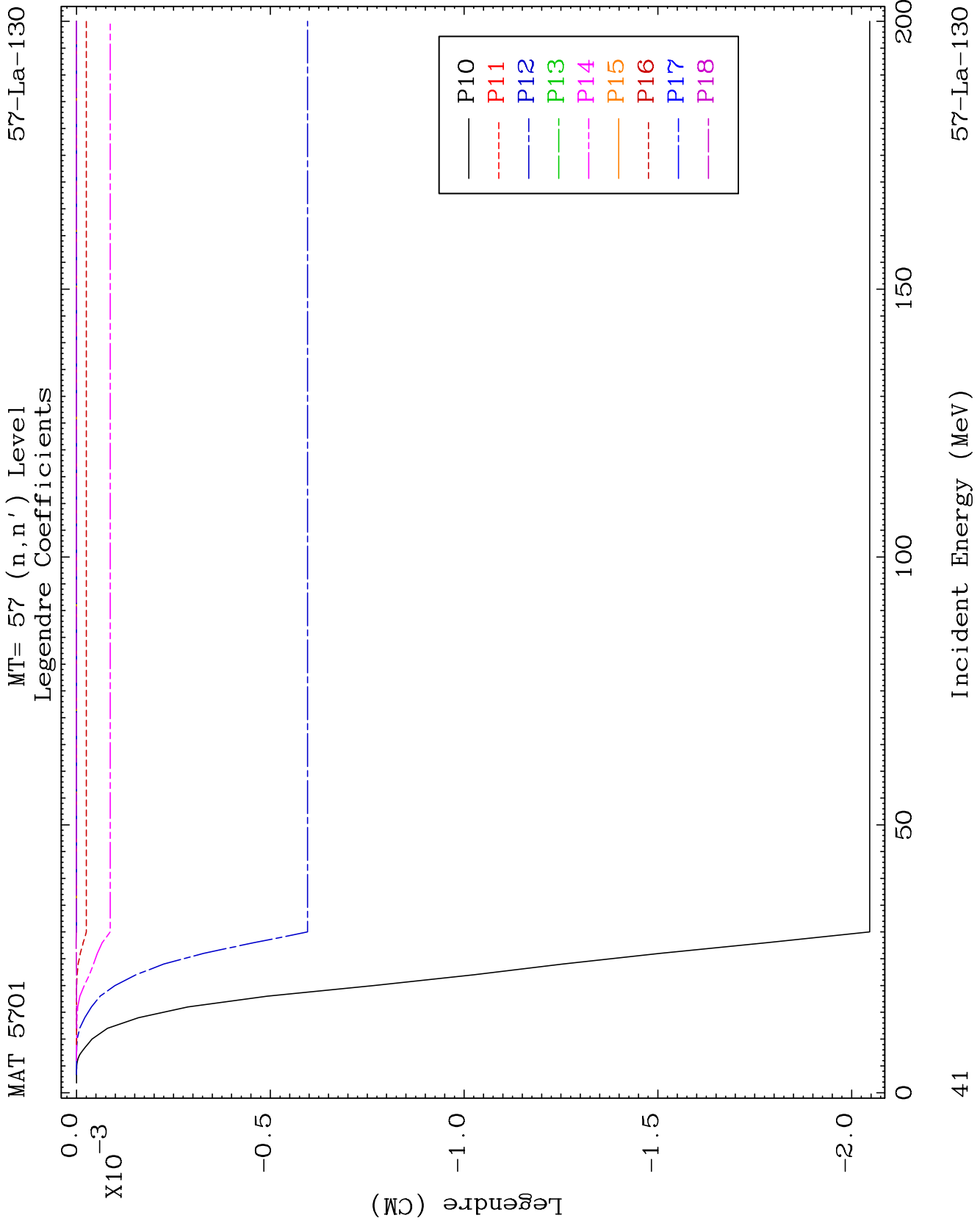


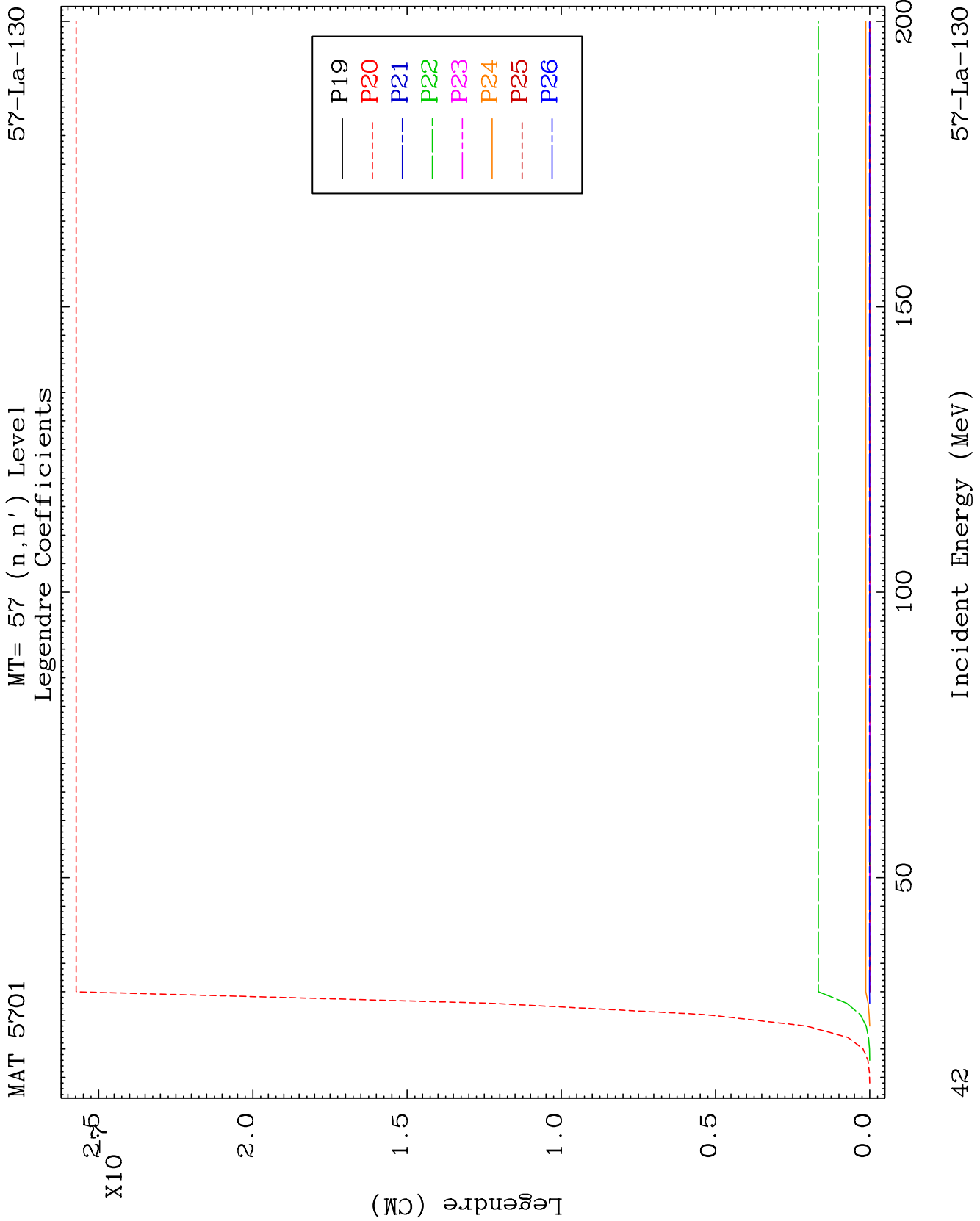




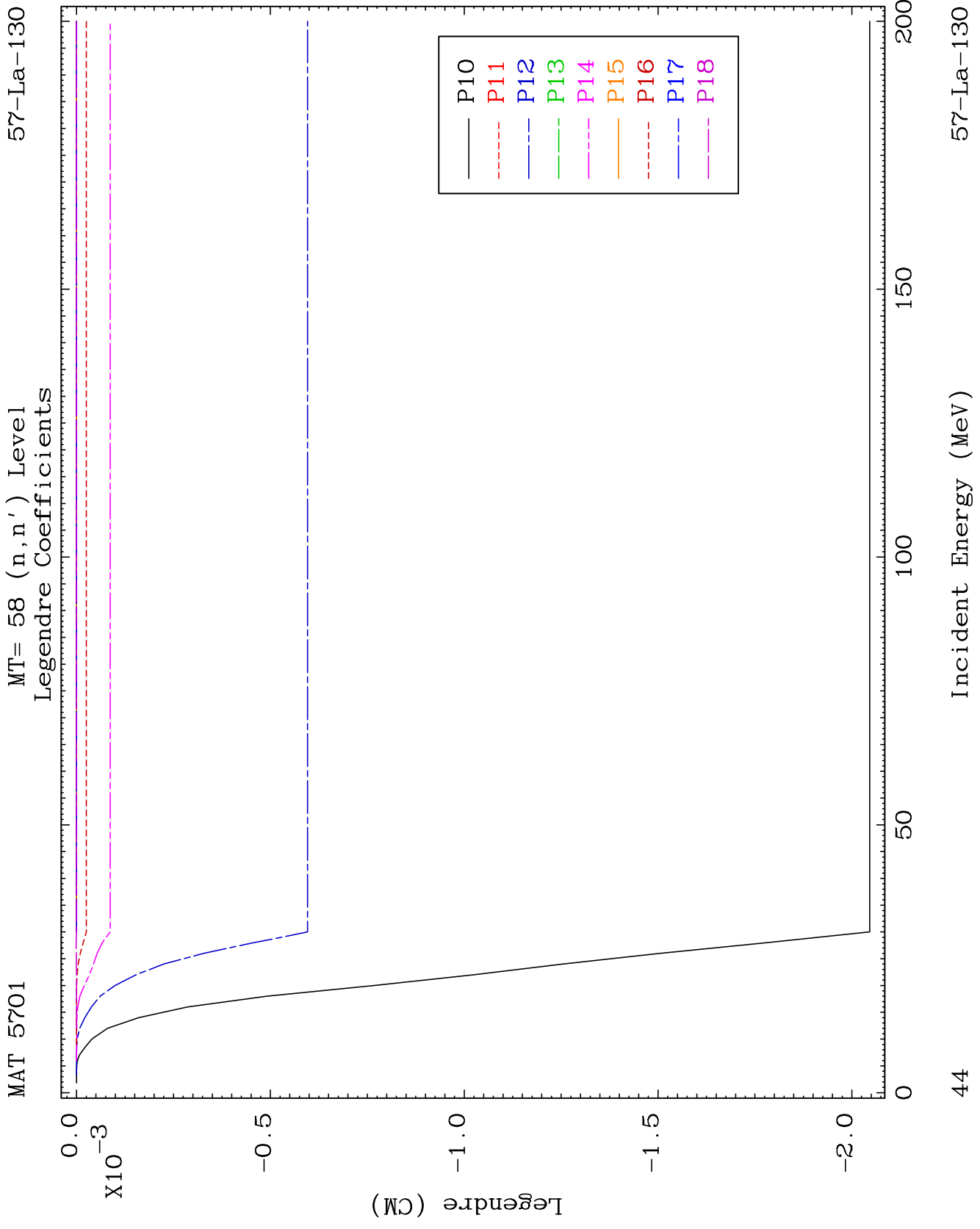










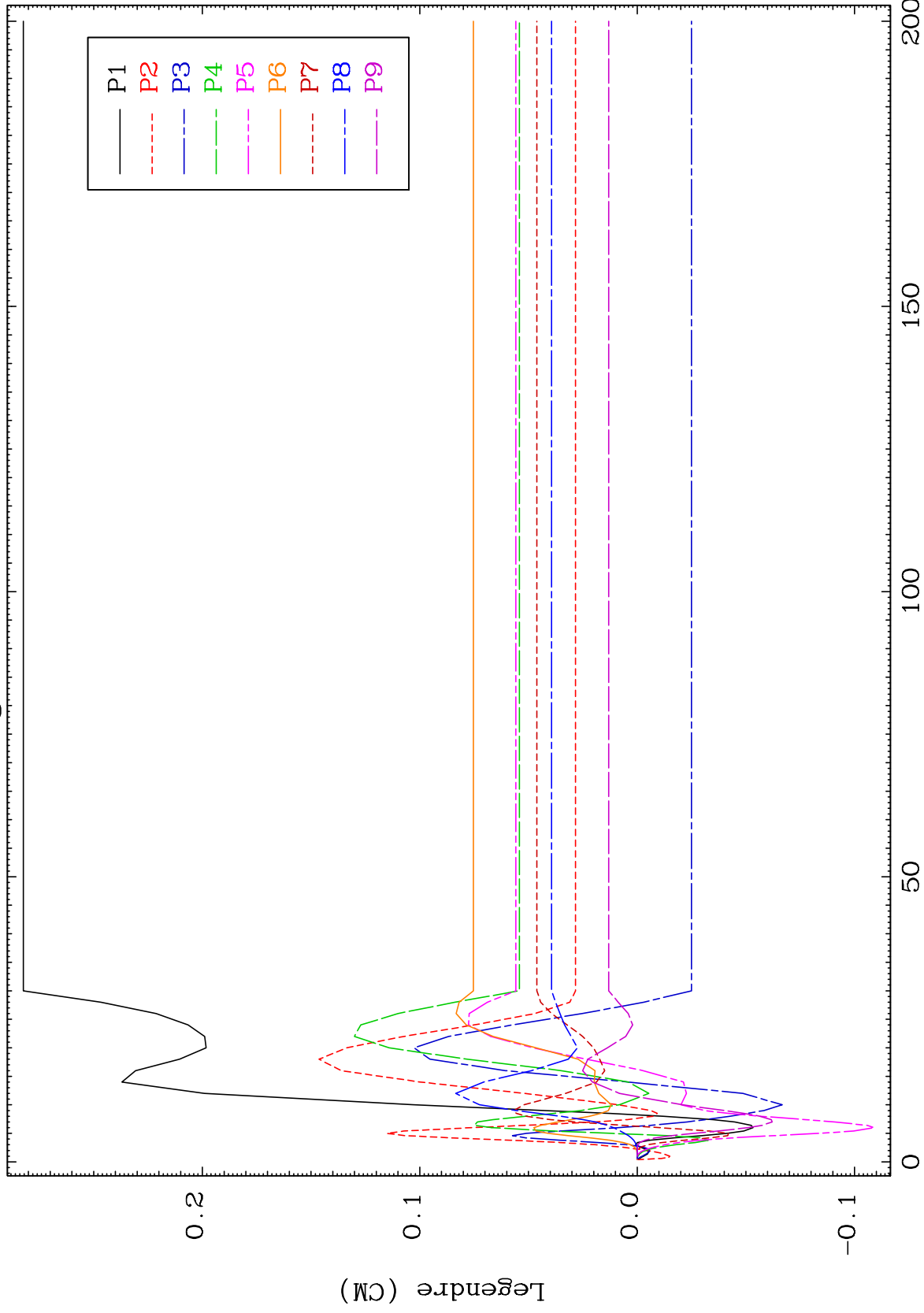




MAT 5701

MT= 59 (n,n') Level  
Legendre Coefficients

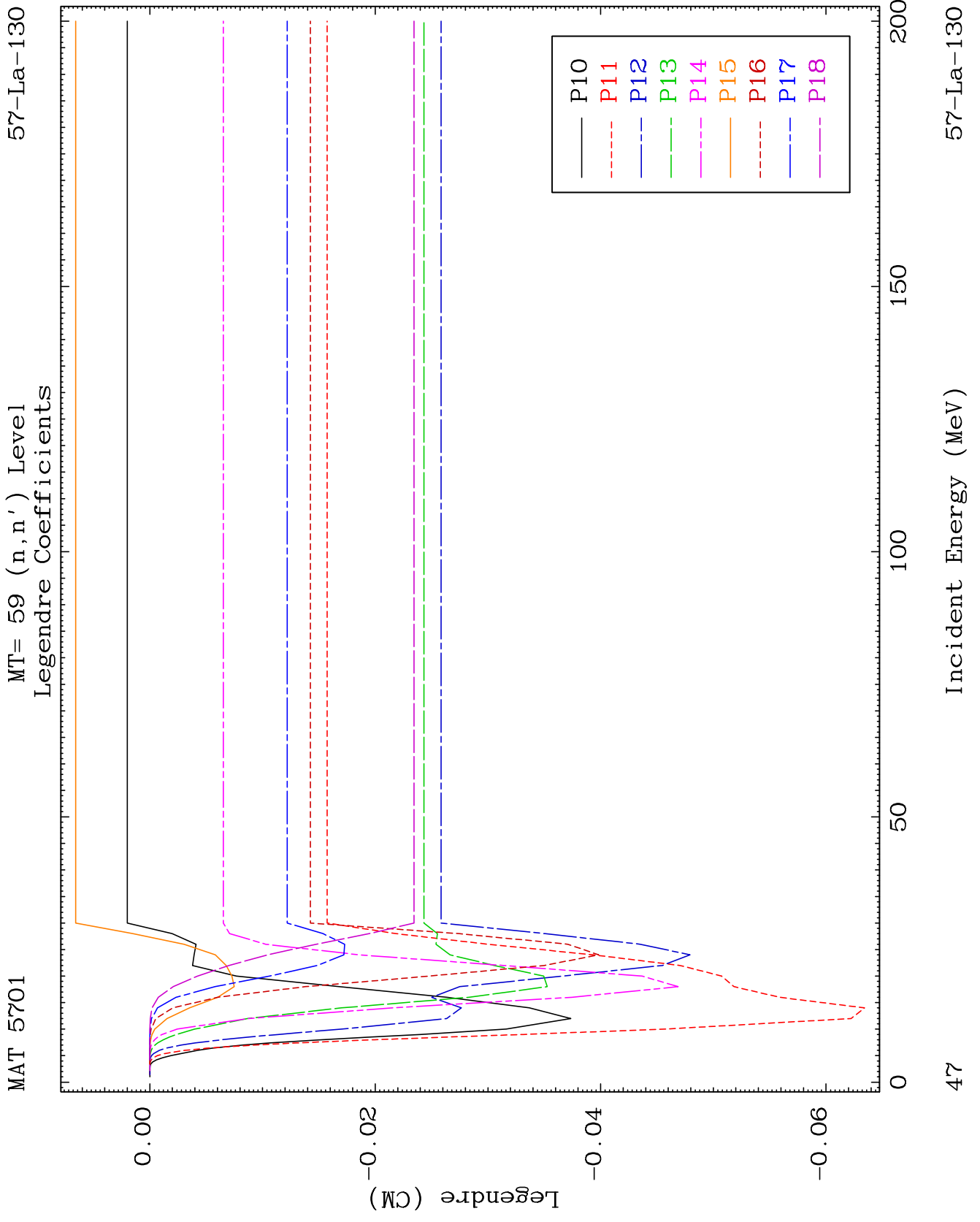
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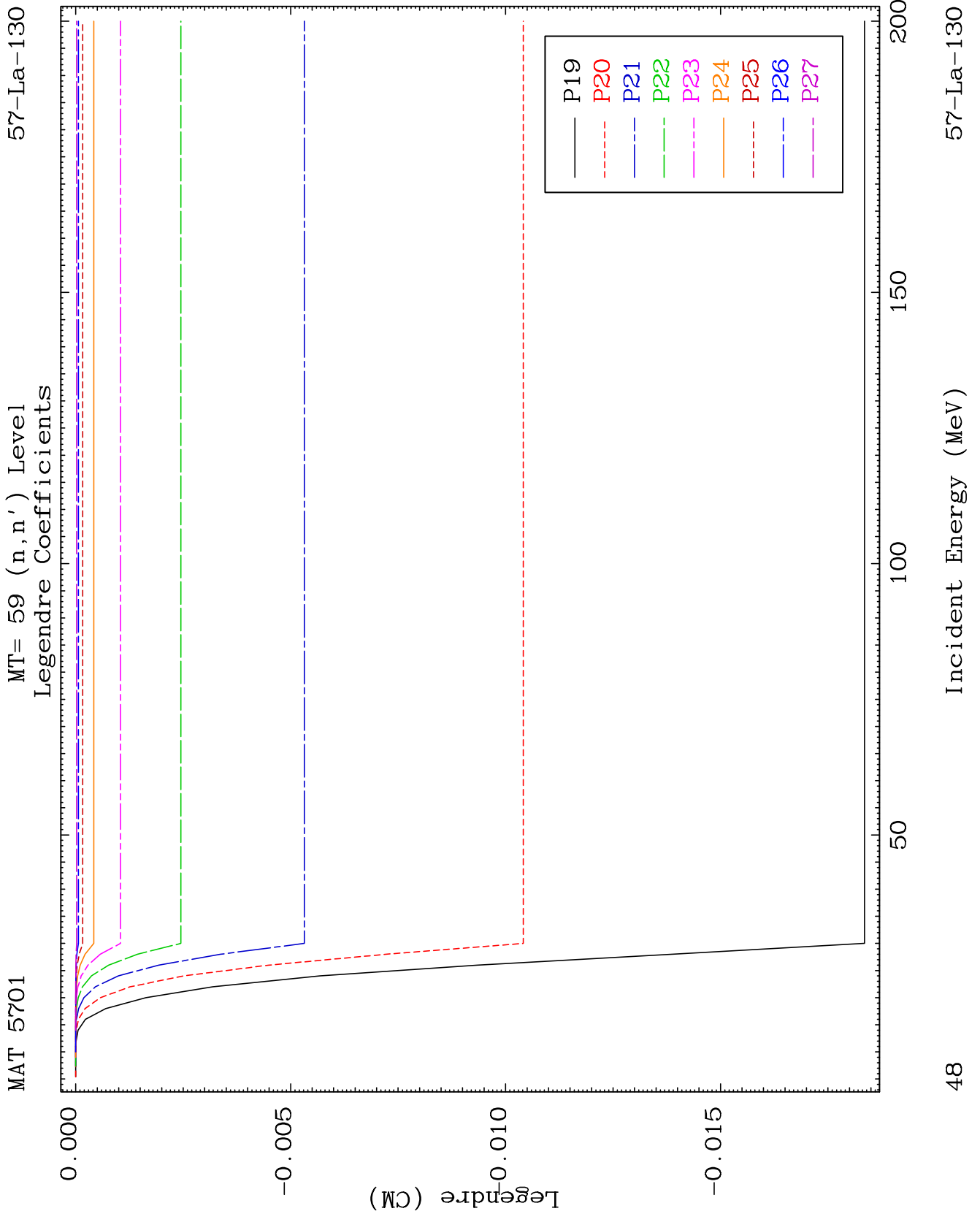
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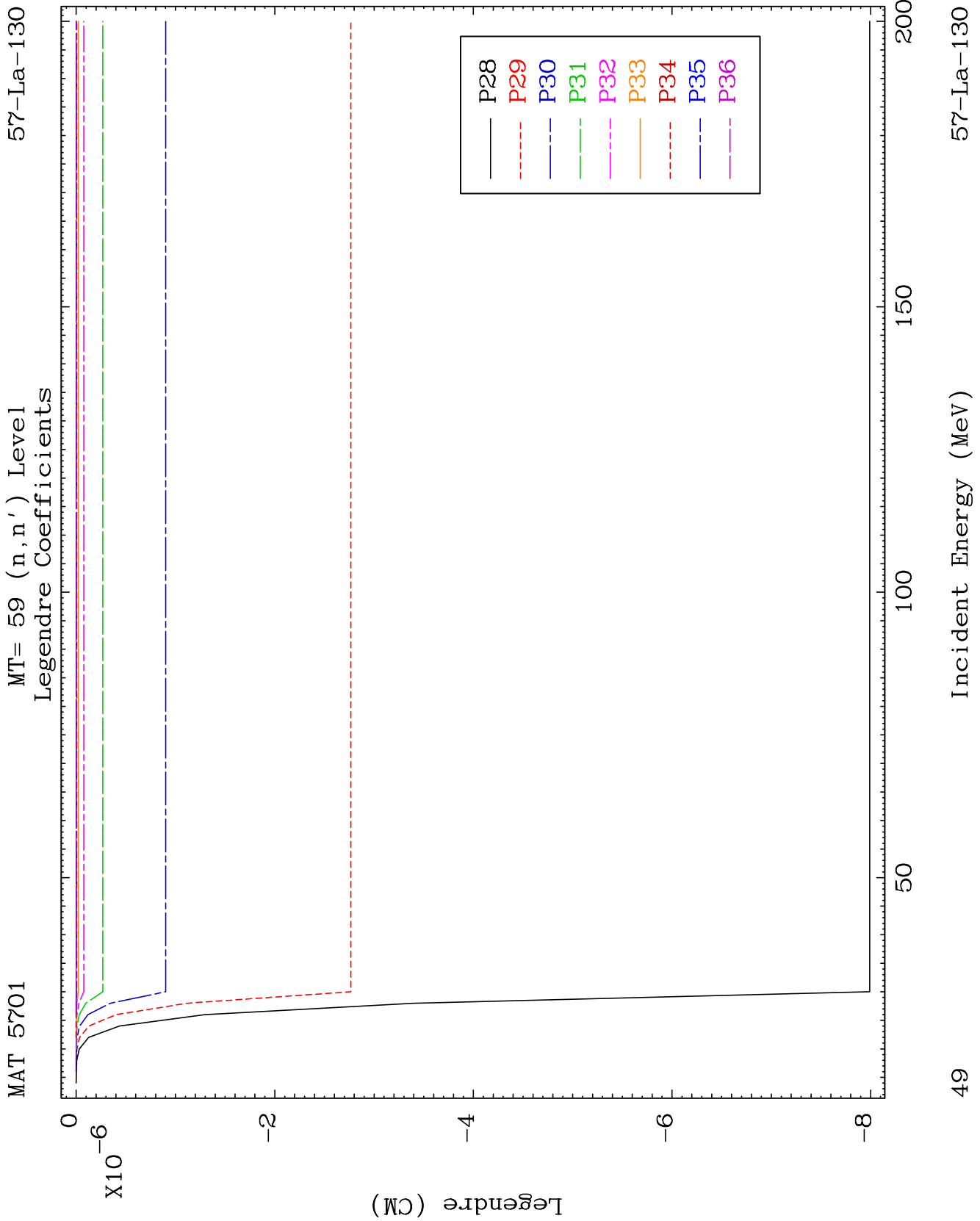
Incident Energy (MeV)

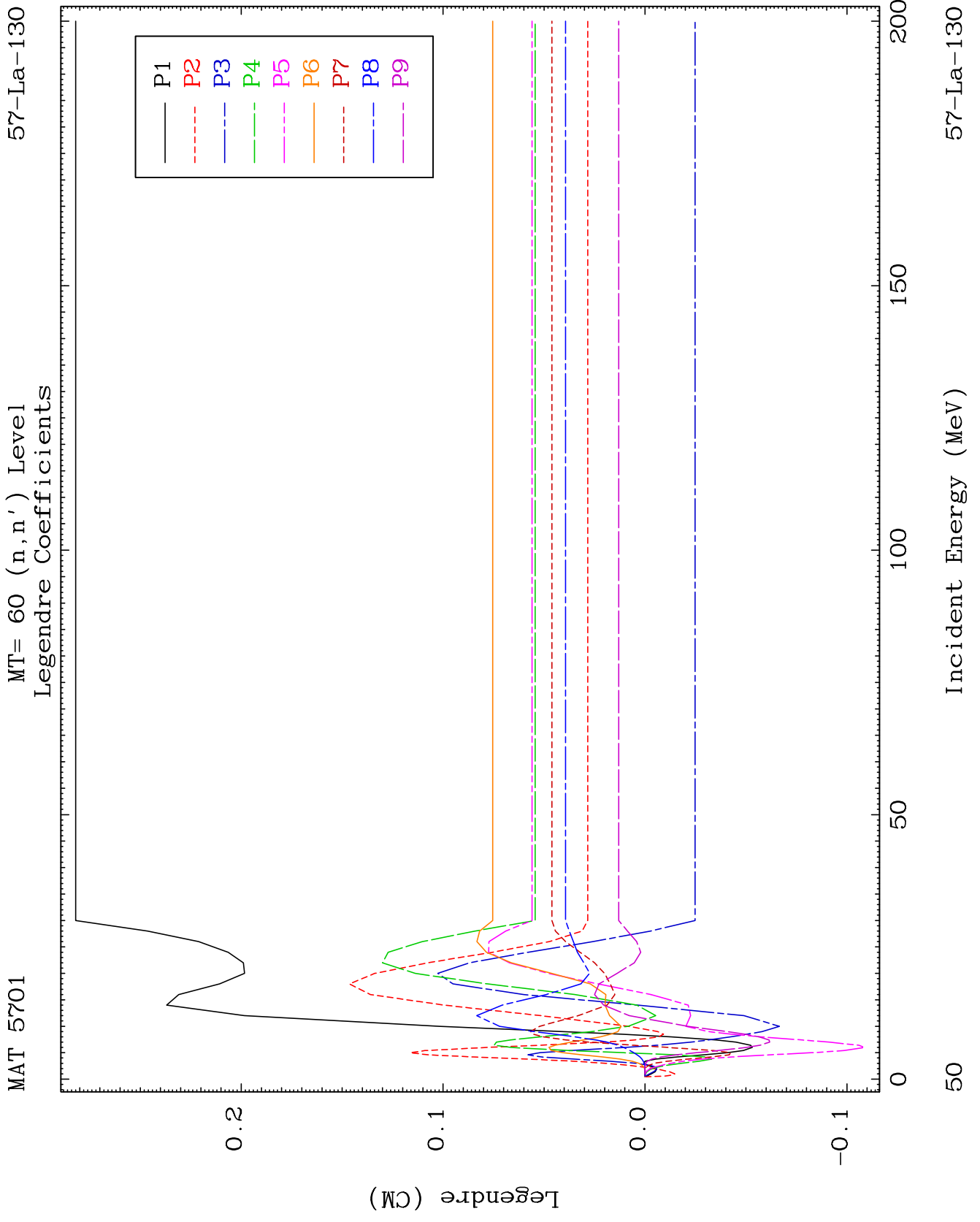
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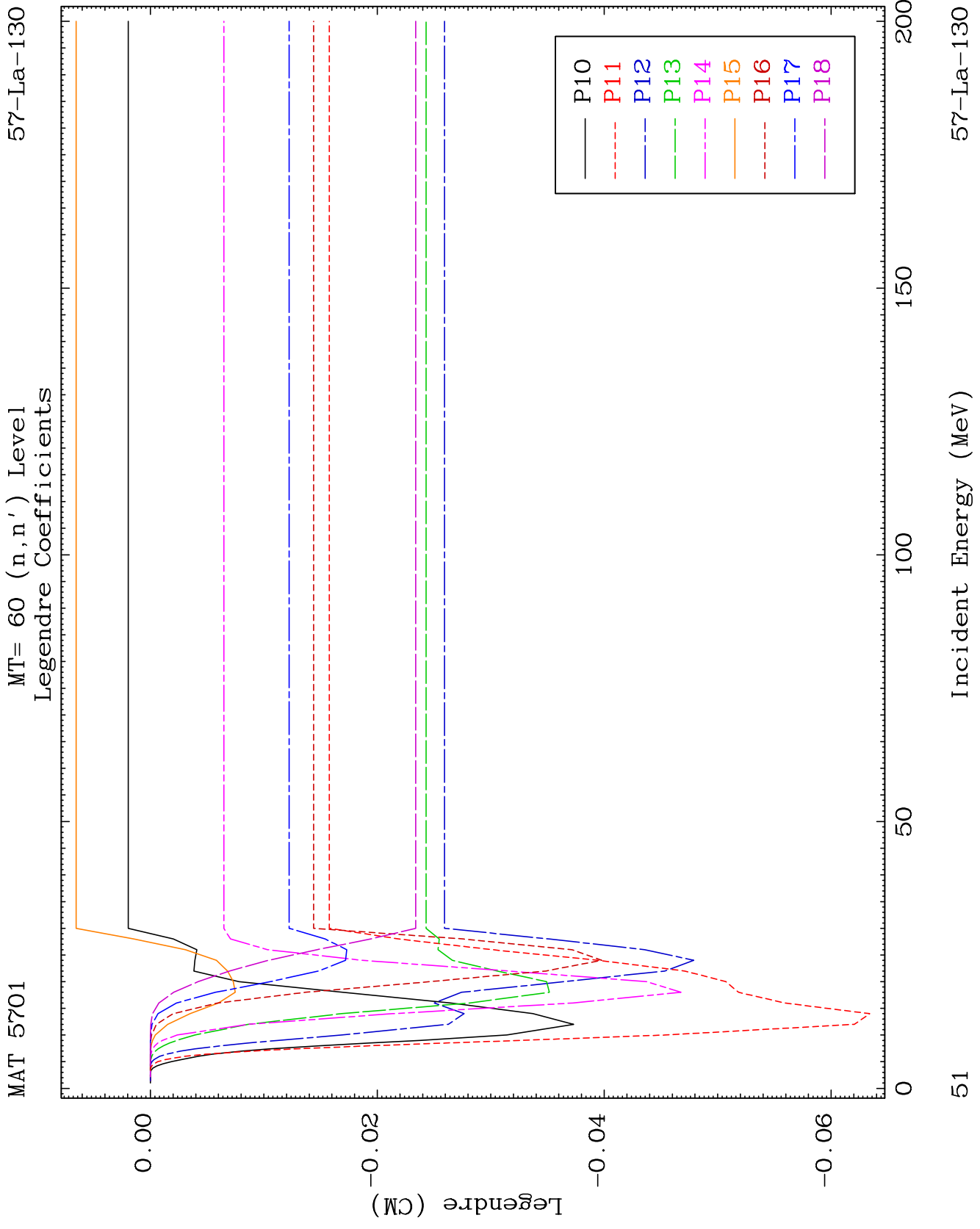


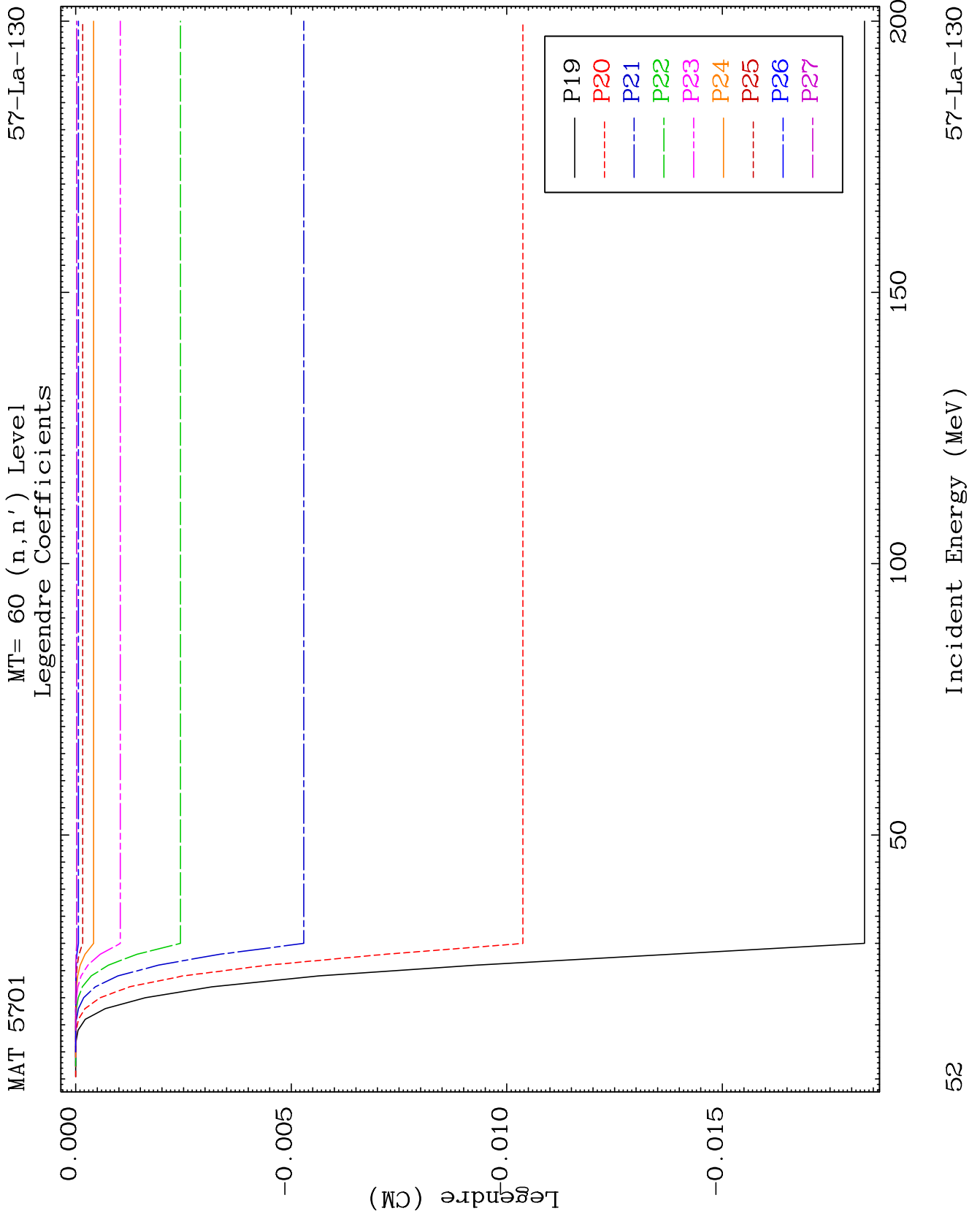


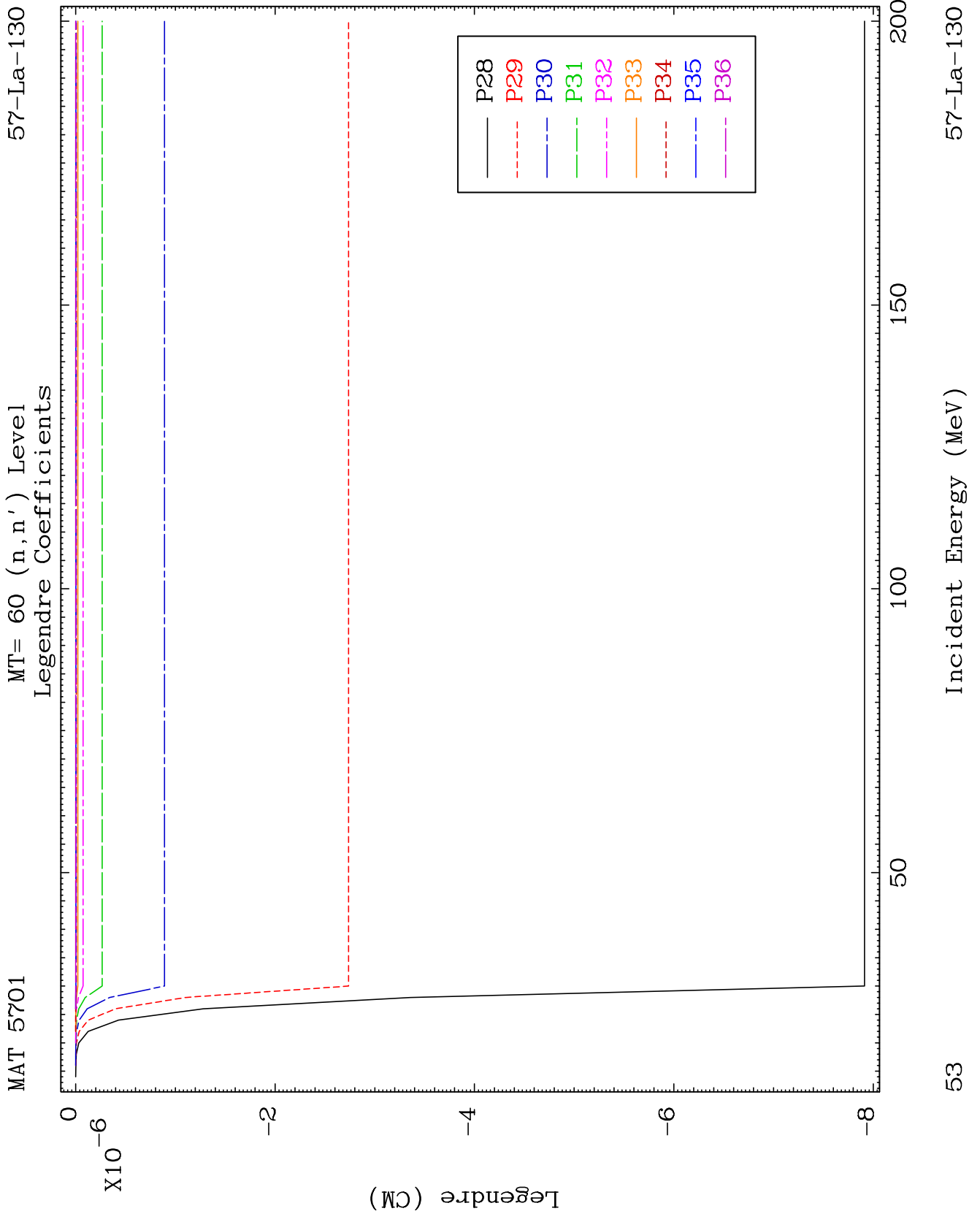




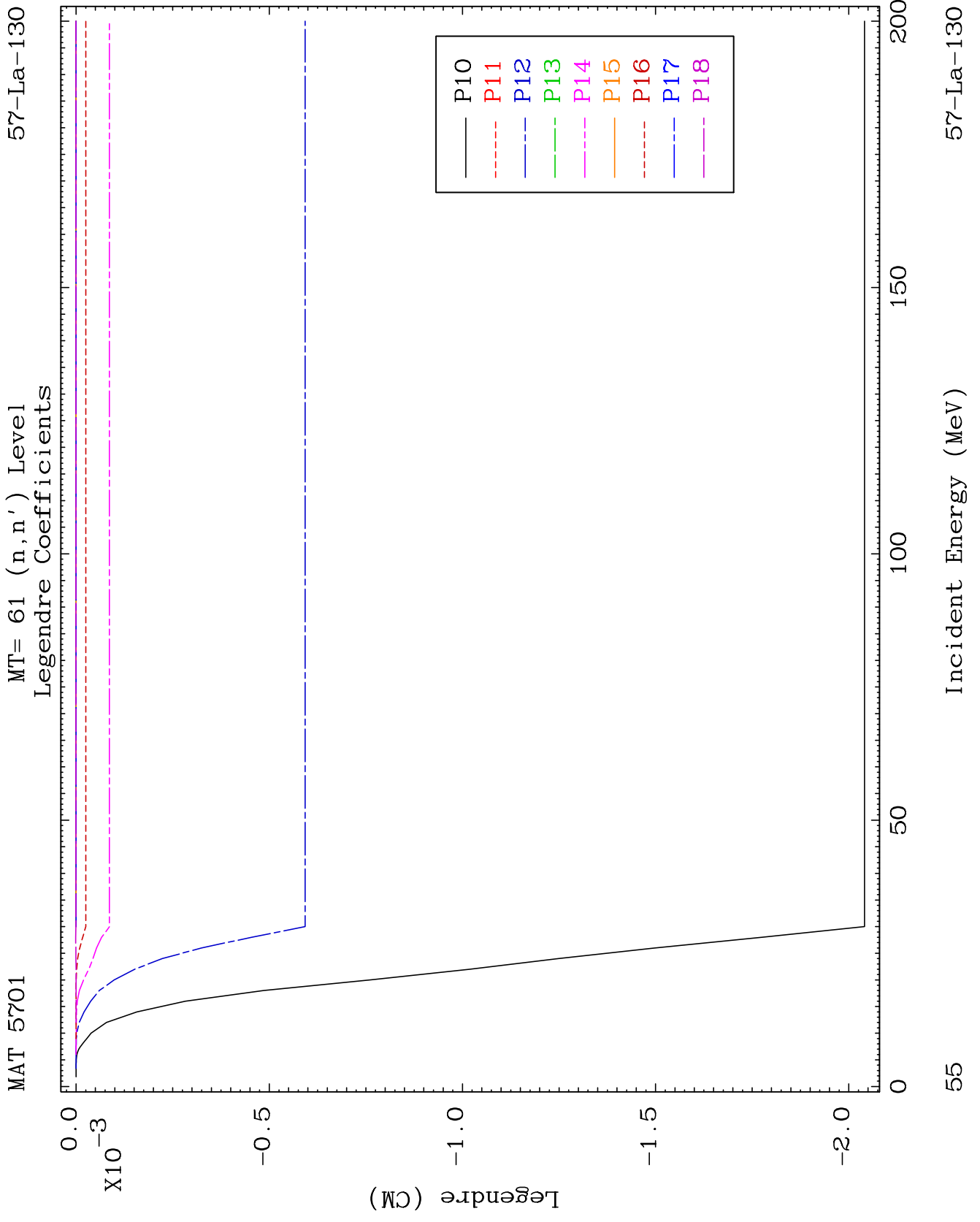




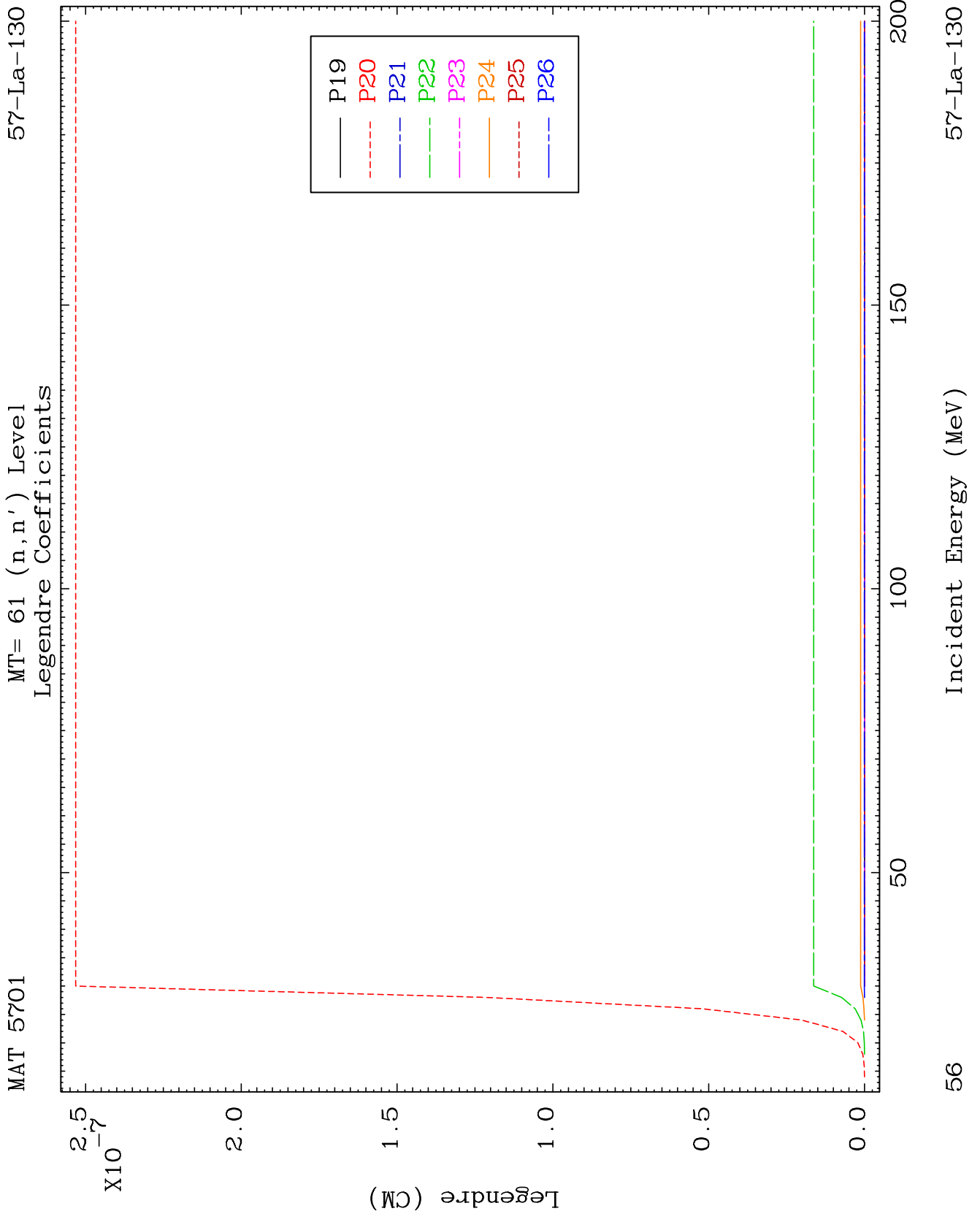








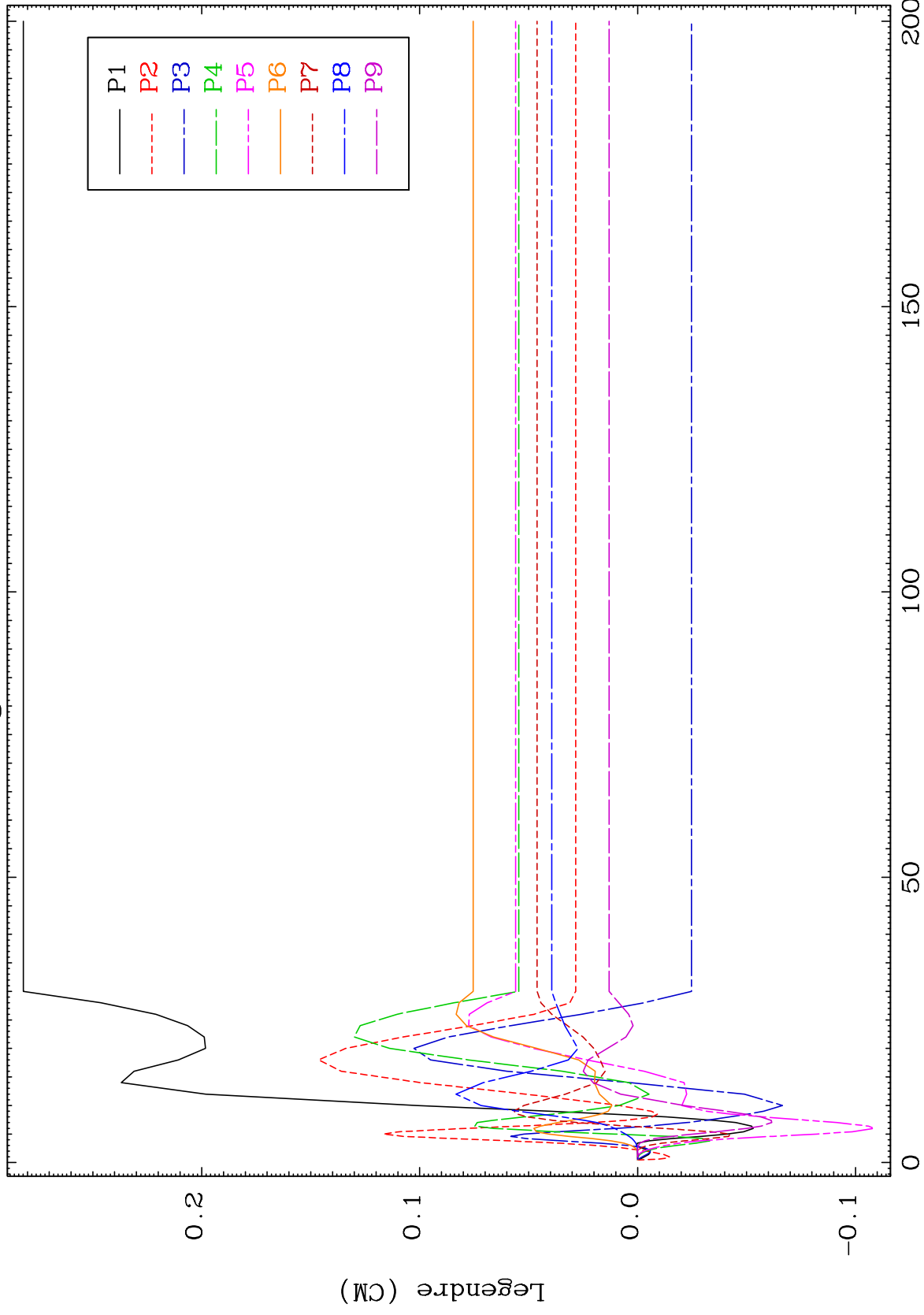




MAT 5701

MT= 62 (n,n') Level  
Legendre Coefficients

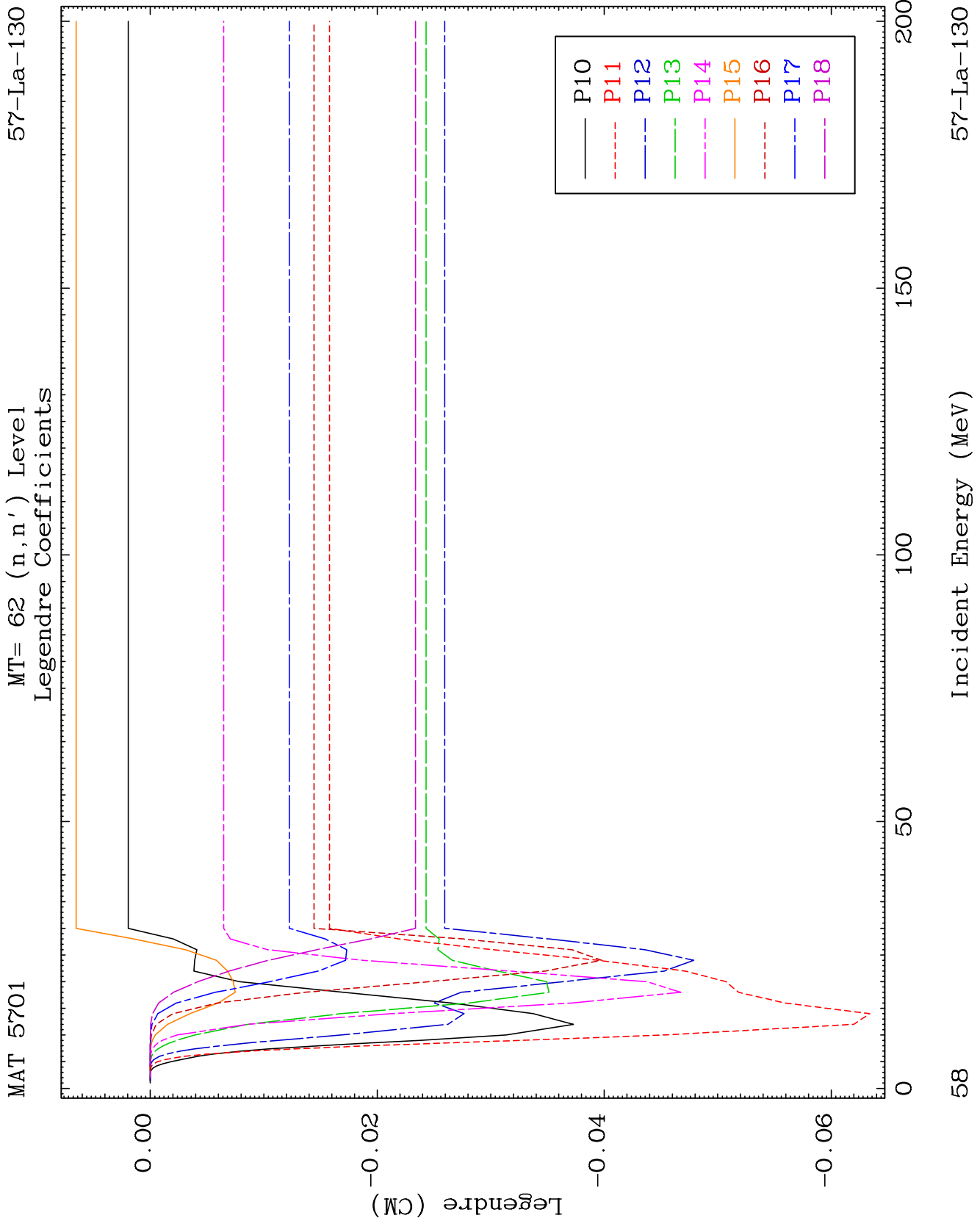
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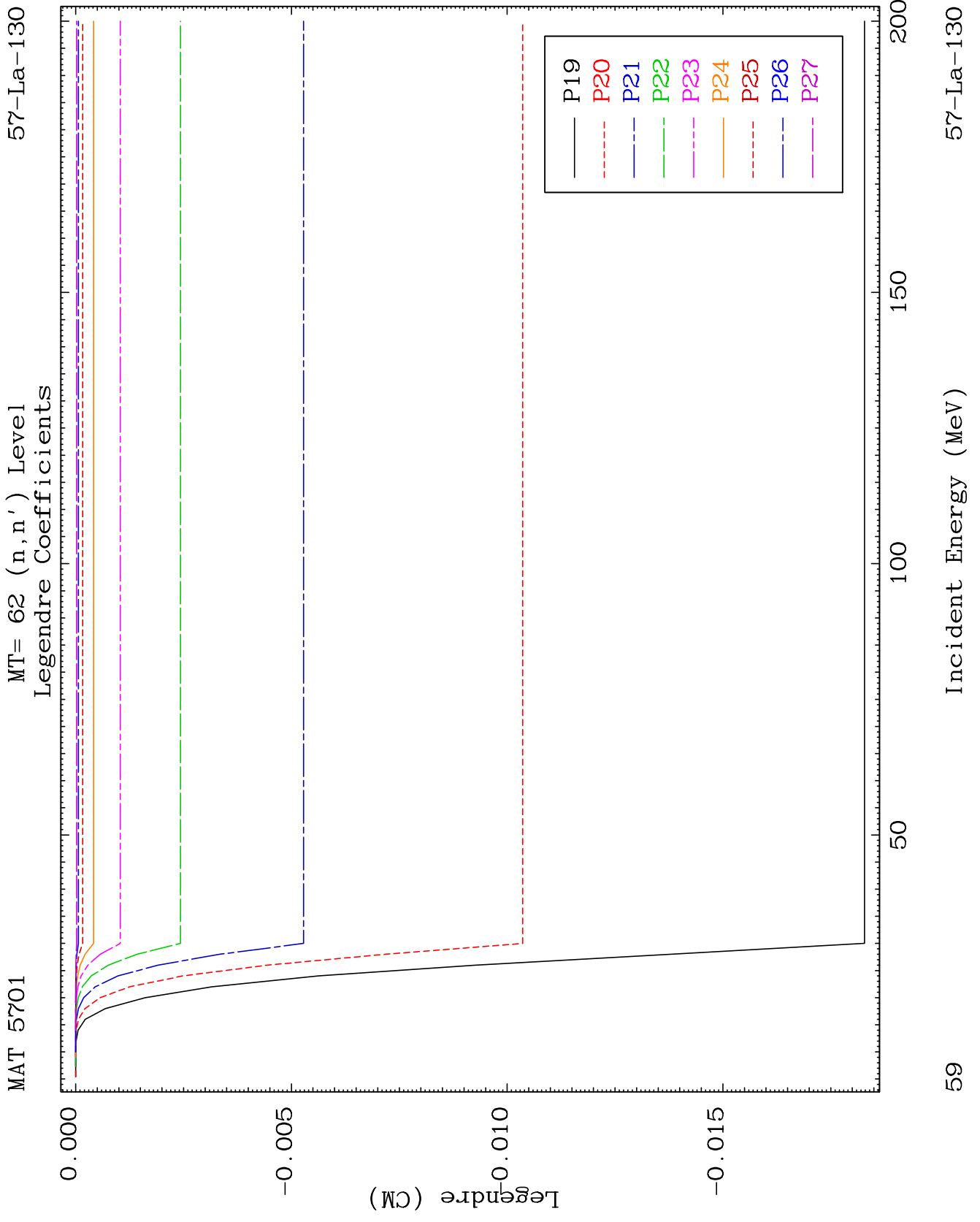


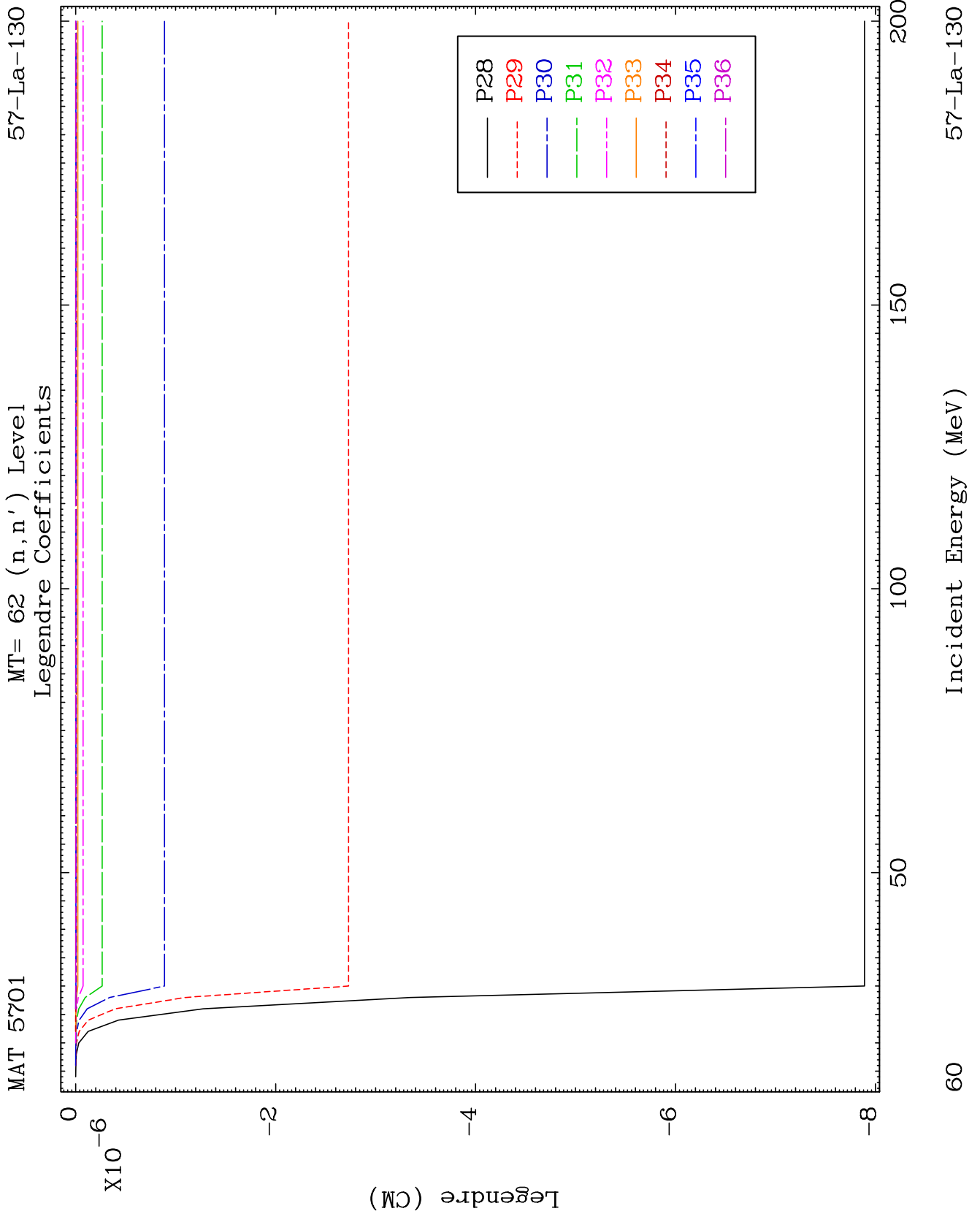
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Incident Energy (MeV)

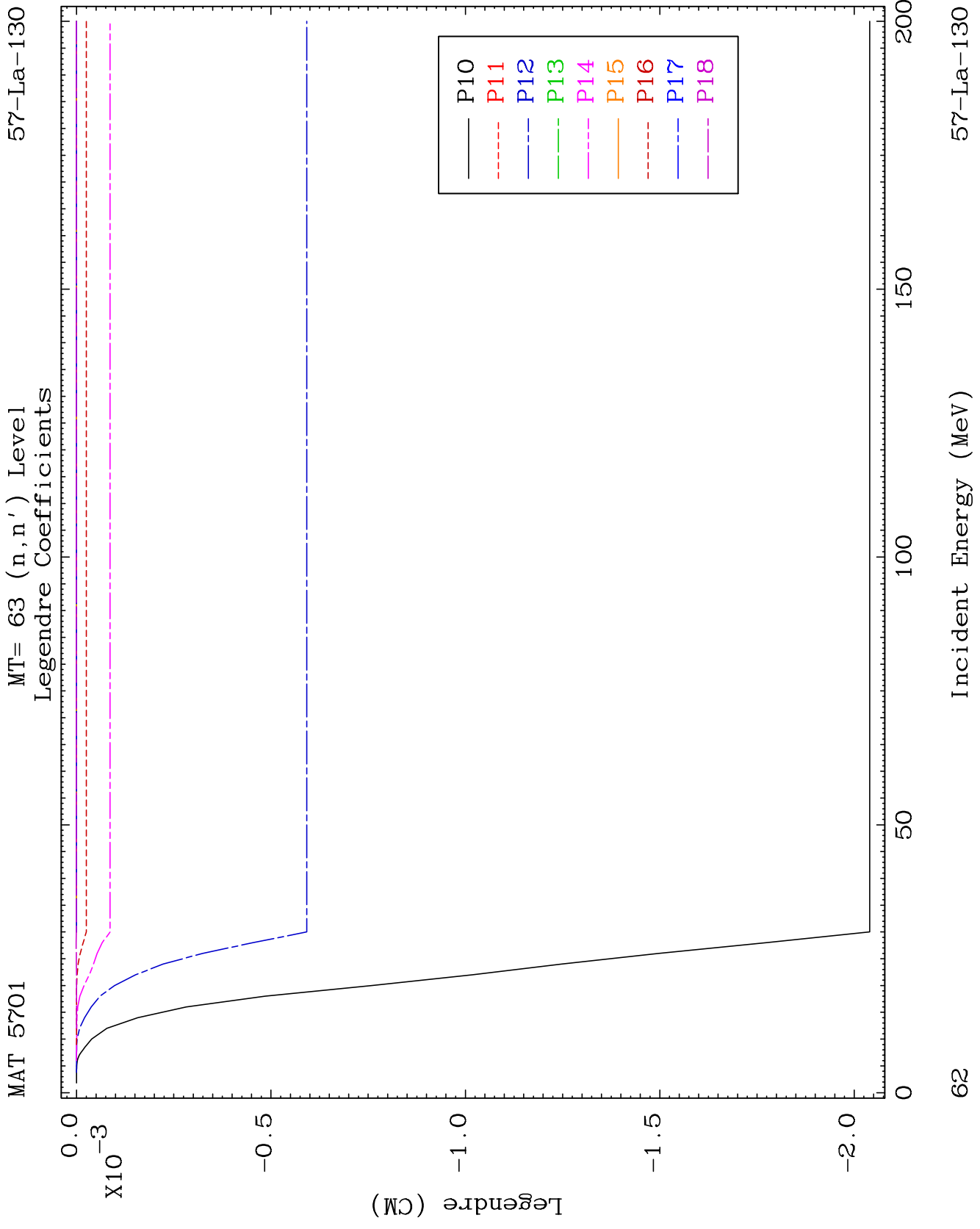
57-La-130







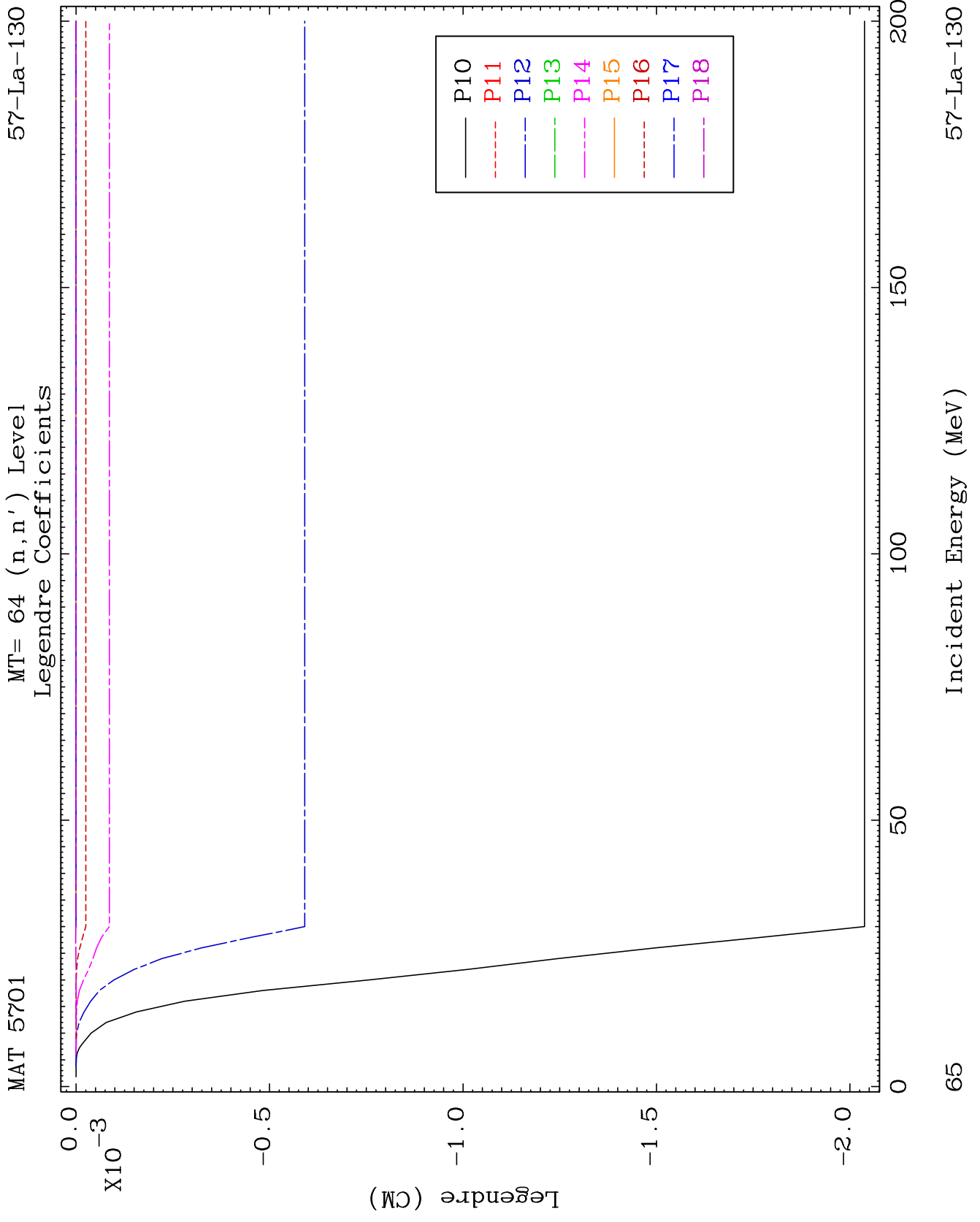








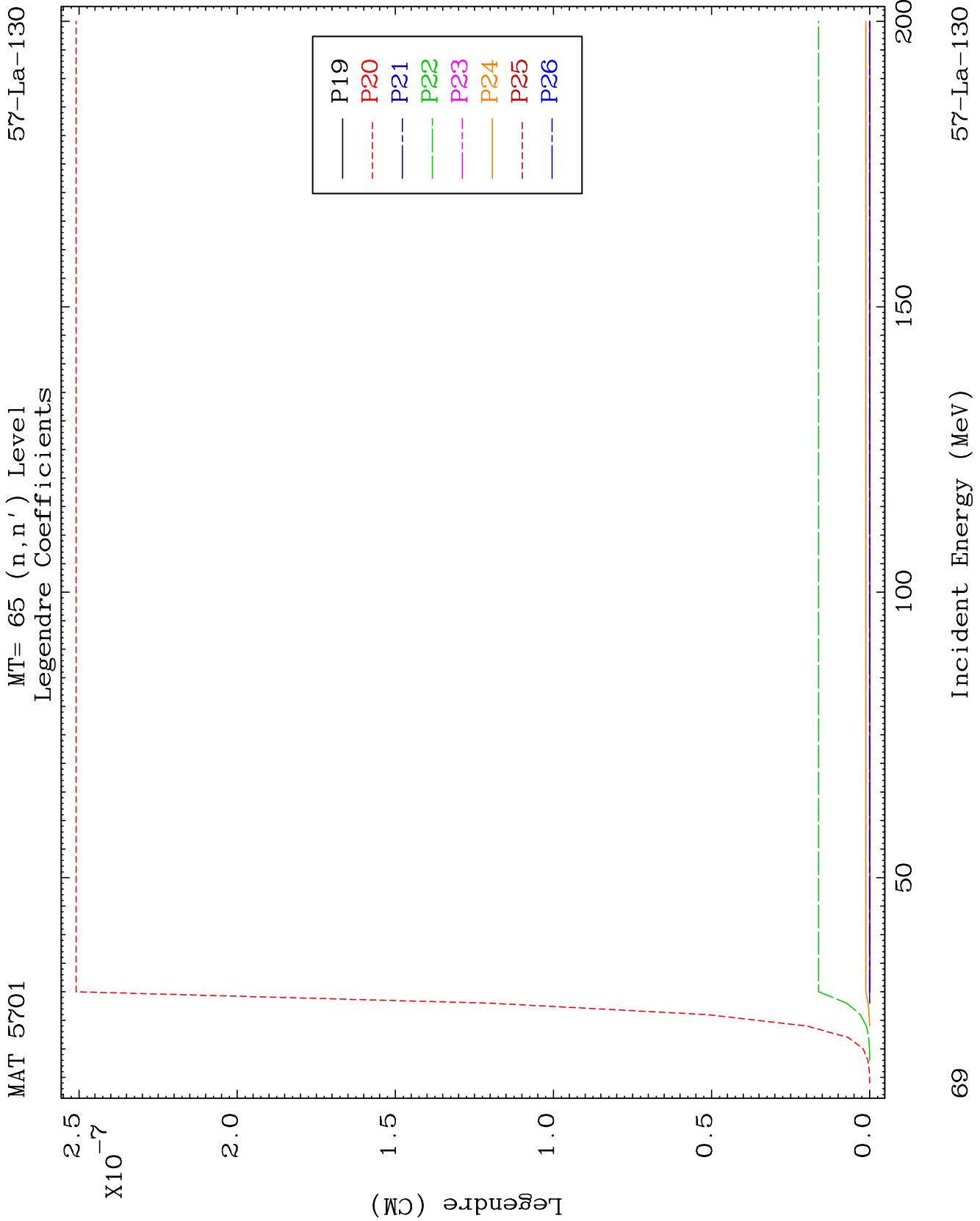








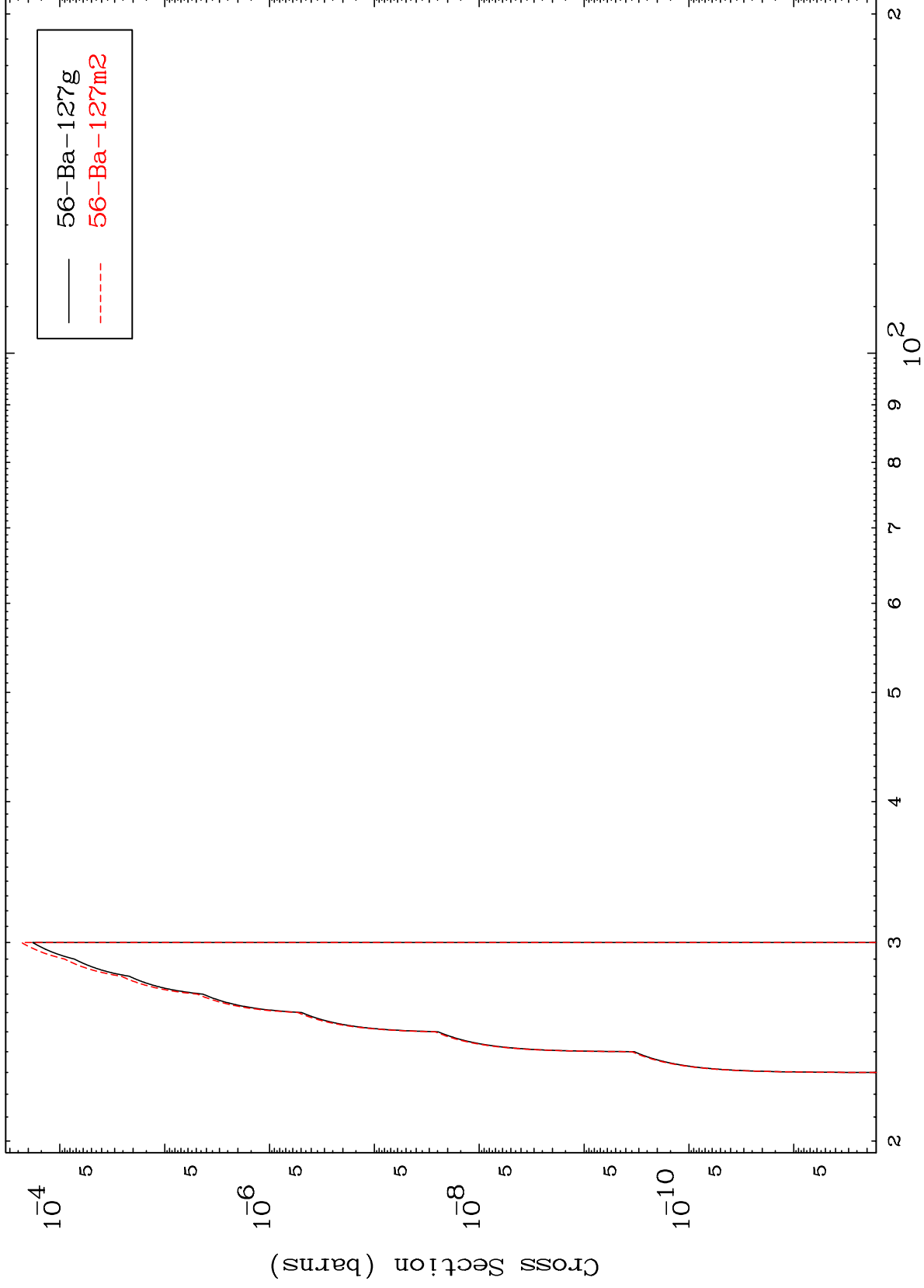




MAT 5701

57-La-130

(n,2n) d  
Radionuclide Production Cross Section



70

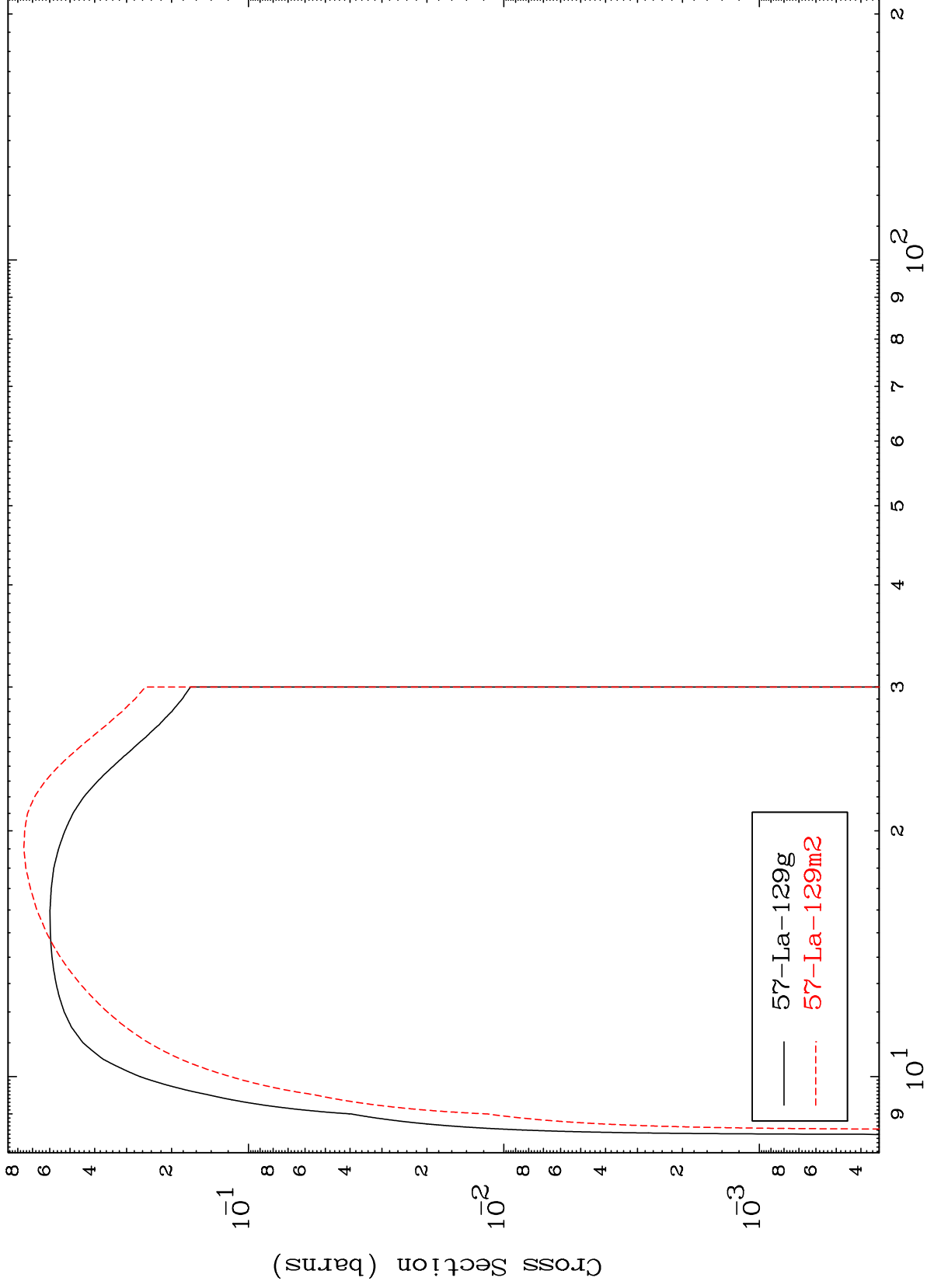
Incident Energy (MeV)

57-La-130

MAT 5701

57-La-130

(n,2n)  
Radionuclide Production Cross Section



57-La-130

Incident Energy (MeV)

71

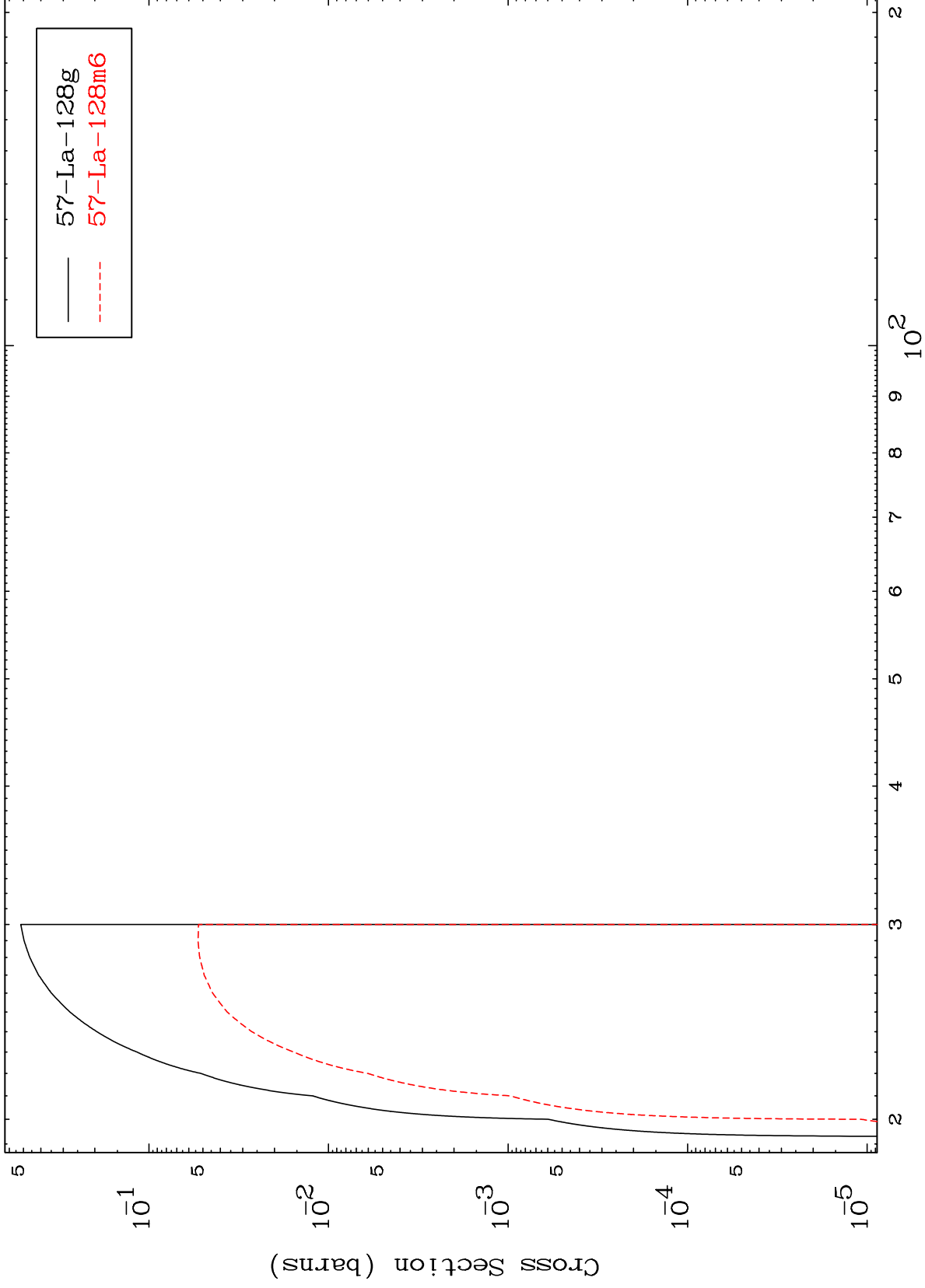


MAT 5701

(n,3n)

57-La-130

Radionuclide Production Cross Section



72

Incident Energy (MeV)

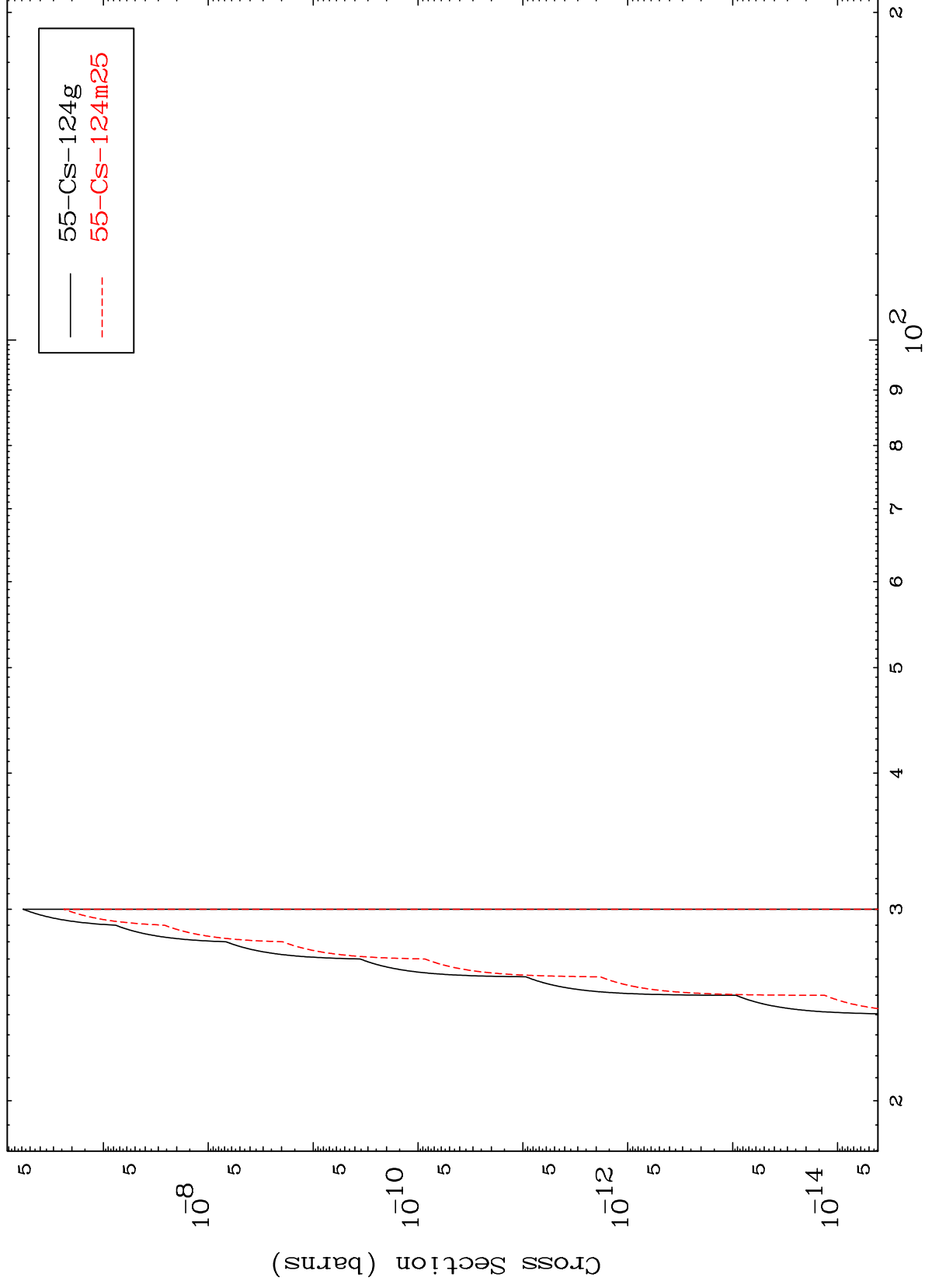
57-La-130

MAT 5701

(n,3n)  $\alpha$

57-La-130

Radionuclide Production Cross Section



73

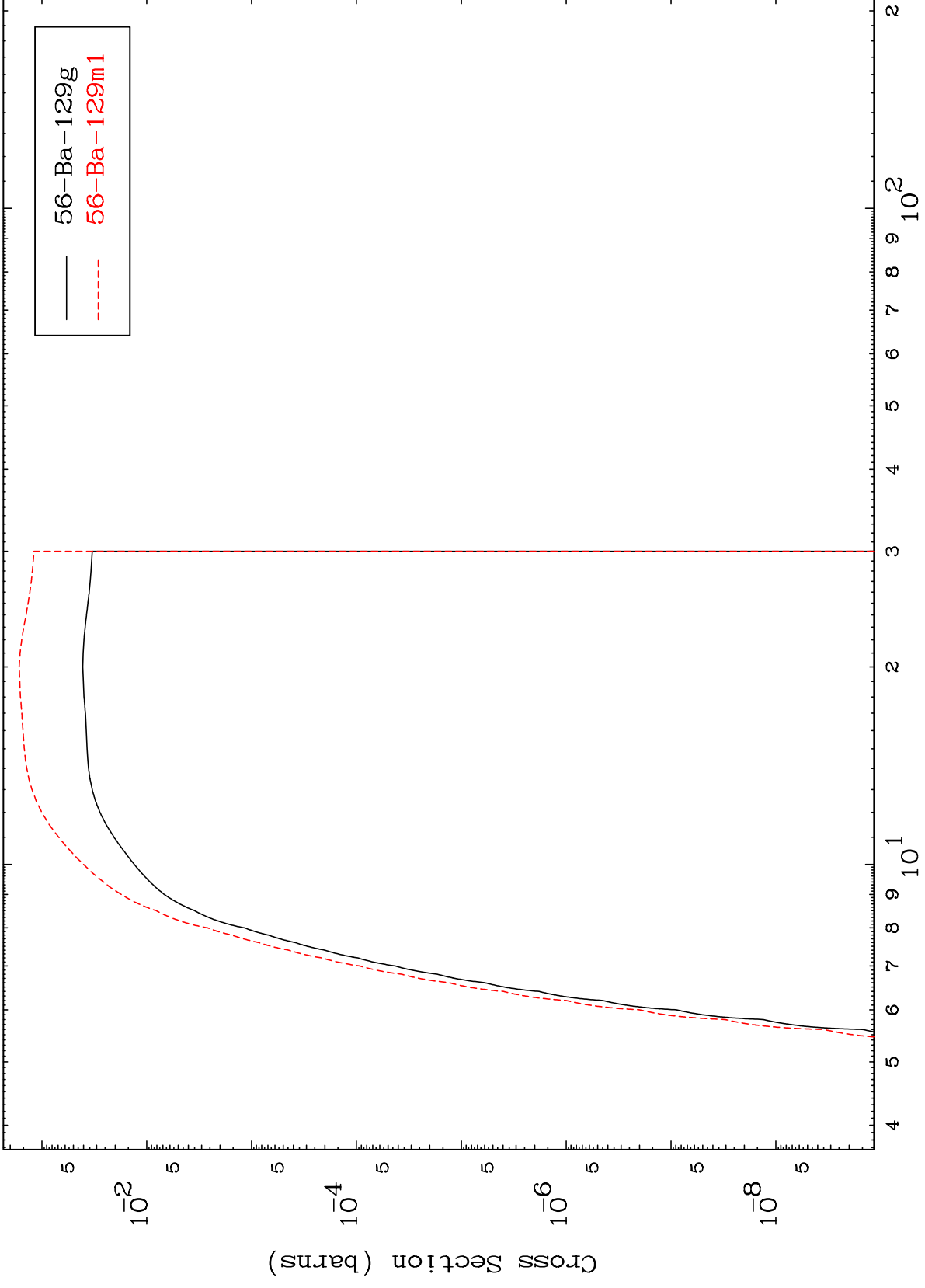
Incident Energy (MeV)

57-La-130

MAT 5701

57-La-130

$(n, n')$  p  
Radionuclide Production Cross Section



74

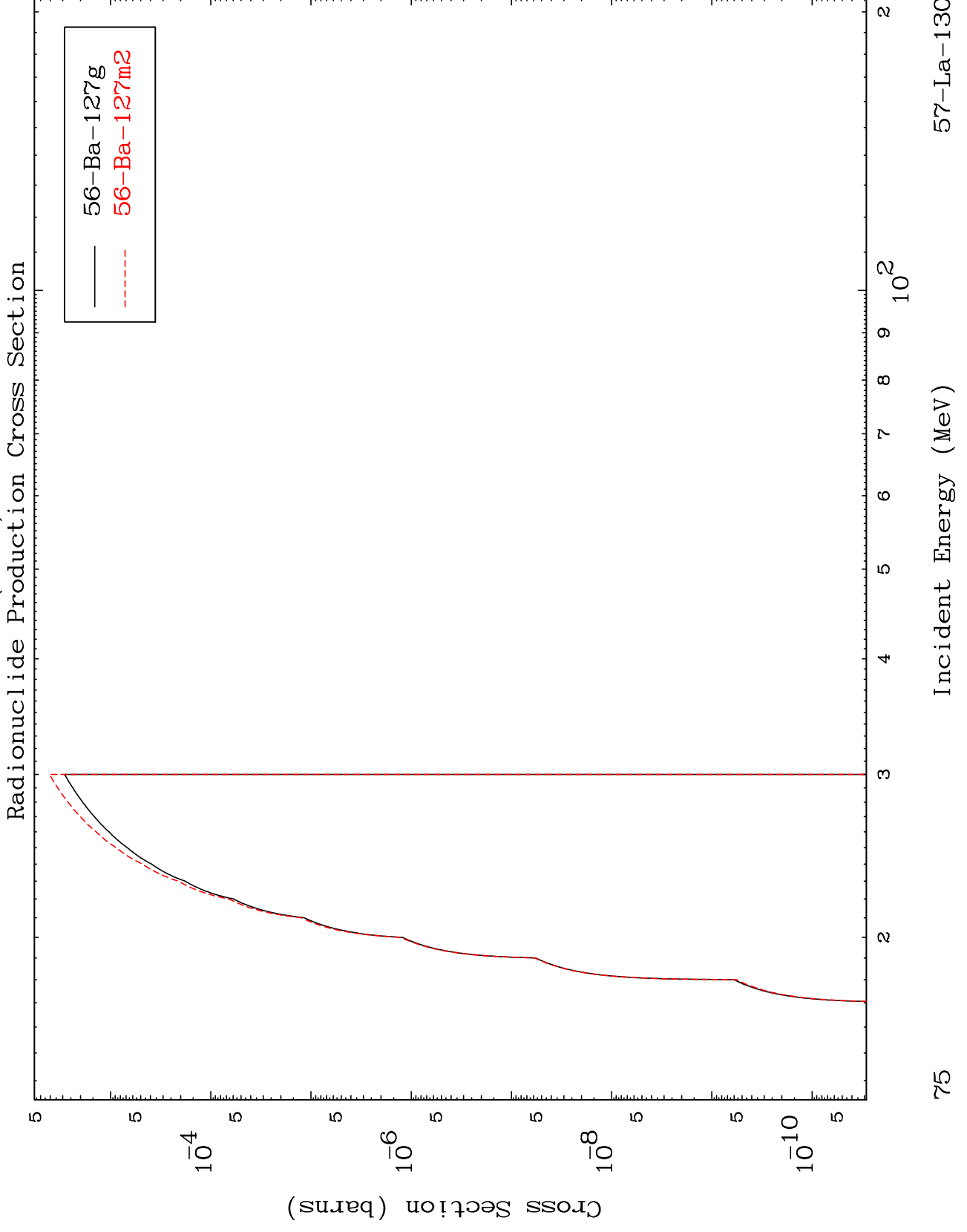
Incident Energy (MeV)

57-La-130

MAT 5701

(n,n') t

57-La-130



75

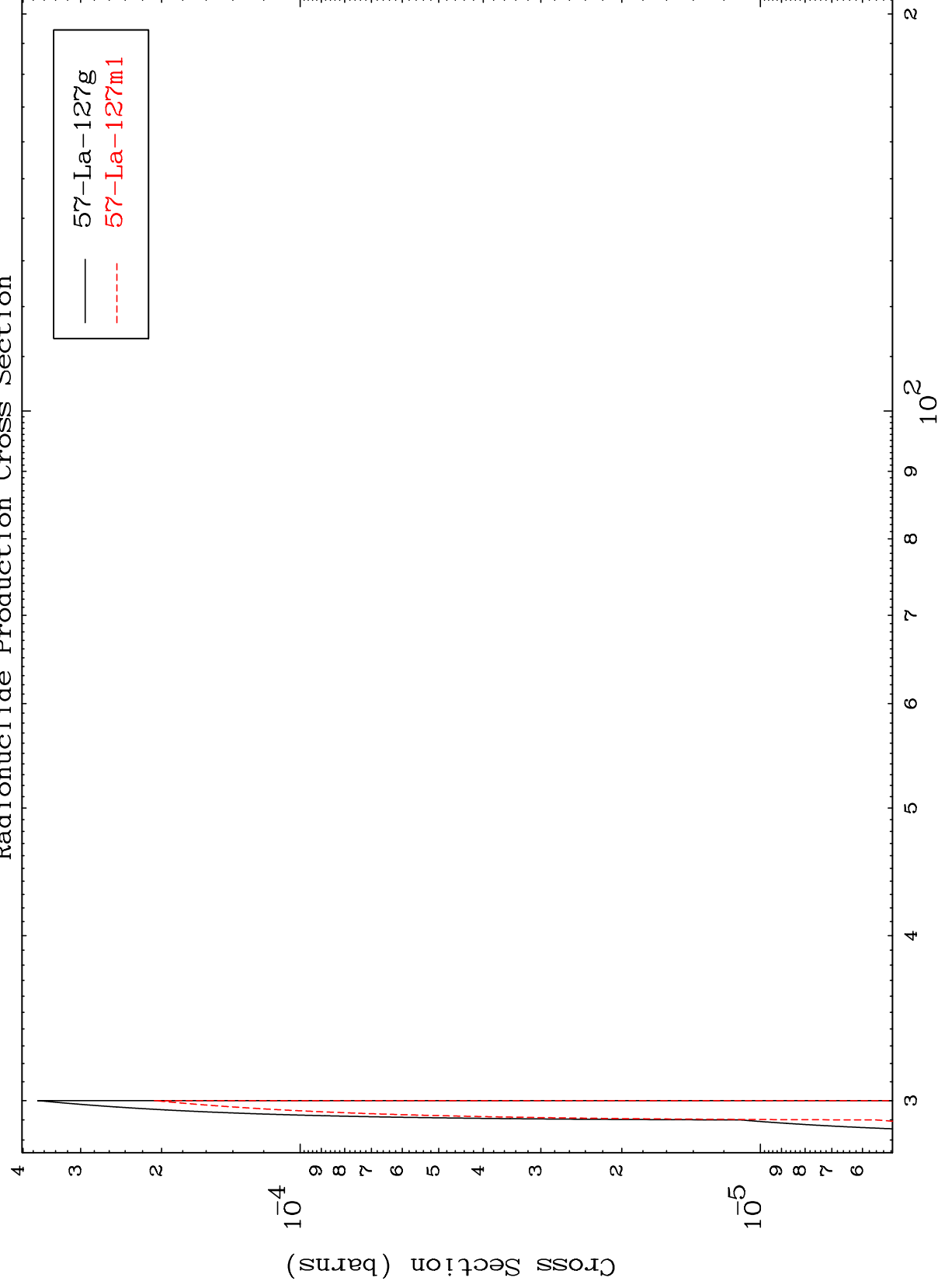
Incident Energy (MeV)

57-La-130

MAT 5701

57-La-130

(n,4n)  
Radionuclide Production Cross Section



76

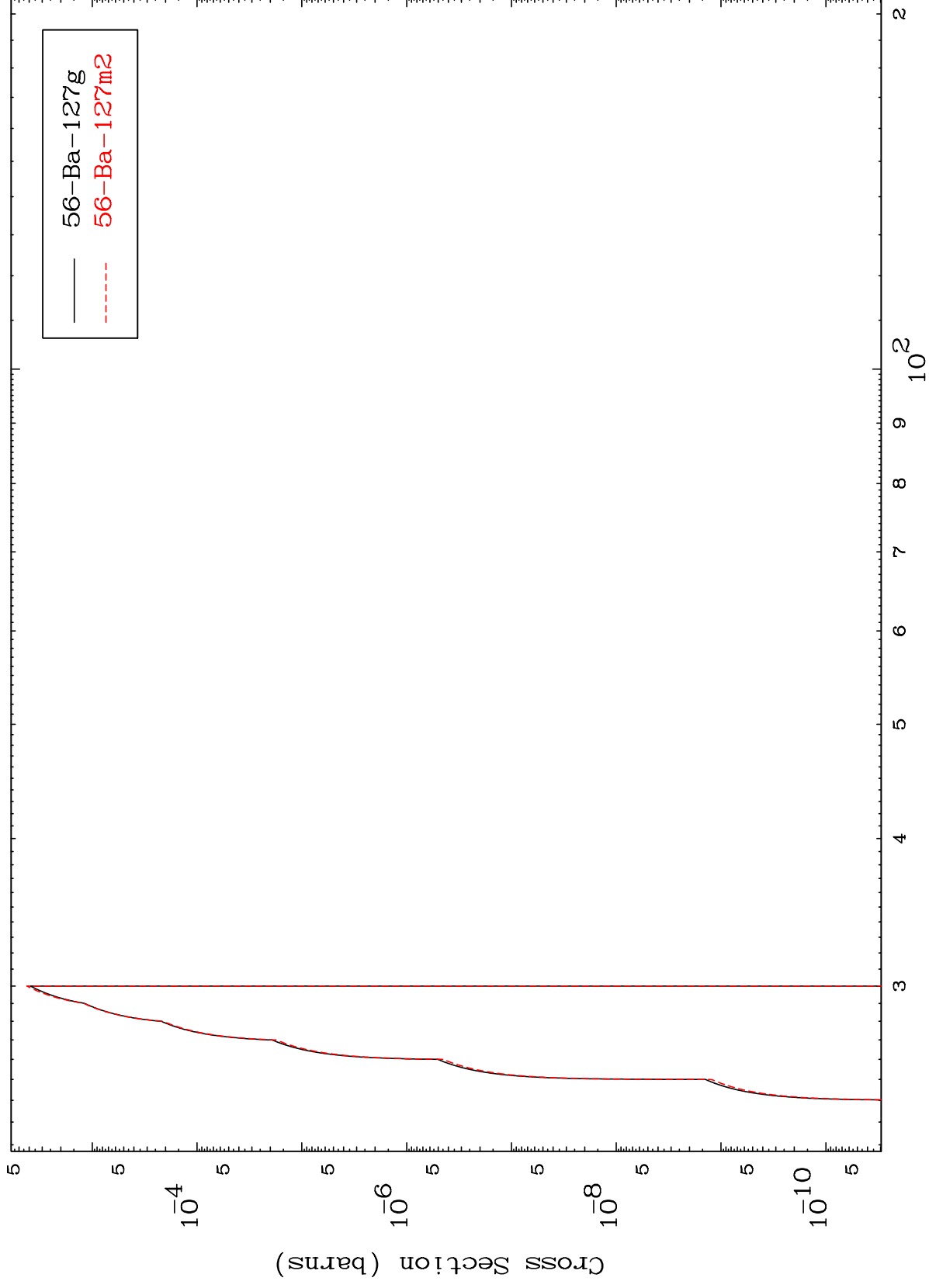
57-La-130

Incident Energy (MeV)

MAT 5701

57-La-130

(n,3n) p  
Radionuclide Production Cross Section



77

57-La-130

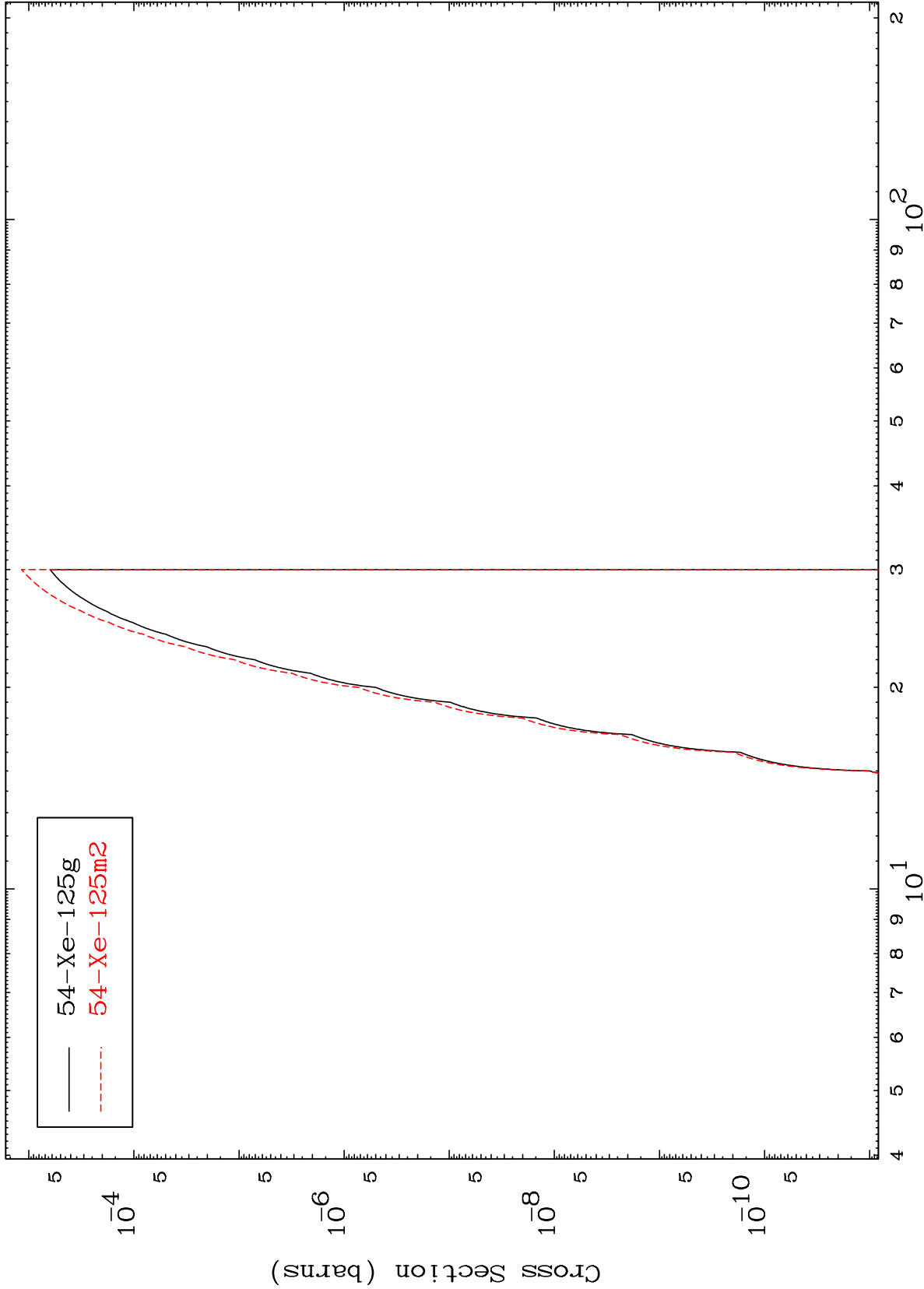
Incident Energy (MeV)

MAT 5701

(n,n') p  $\alpha$

57-La-130

Radionuclide Production Cross Section



78

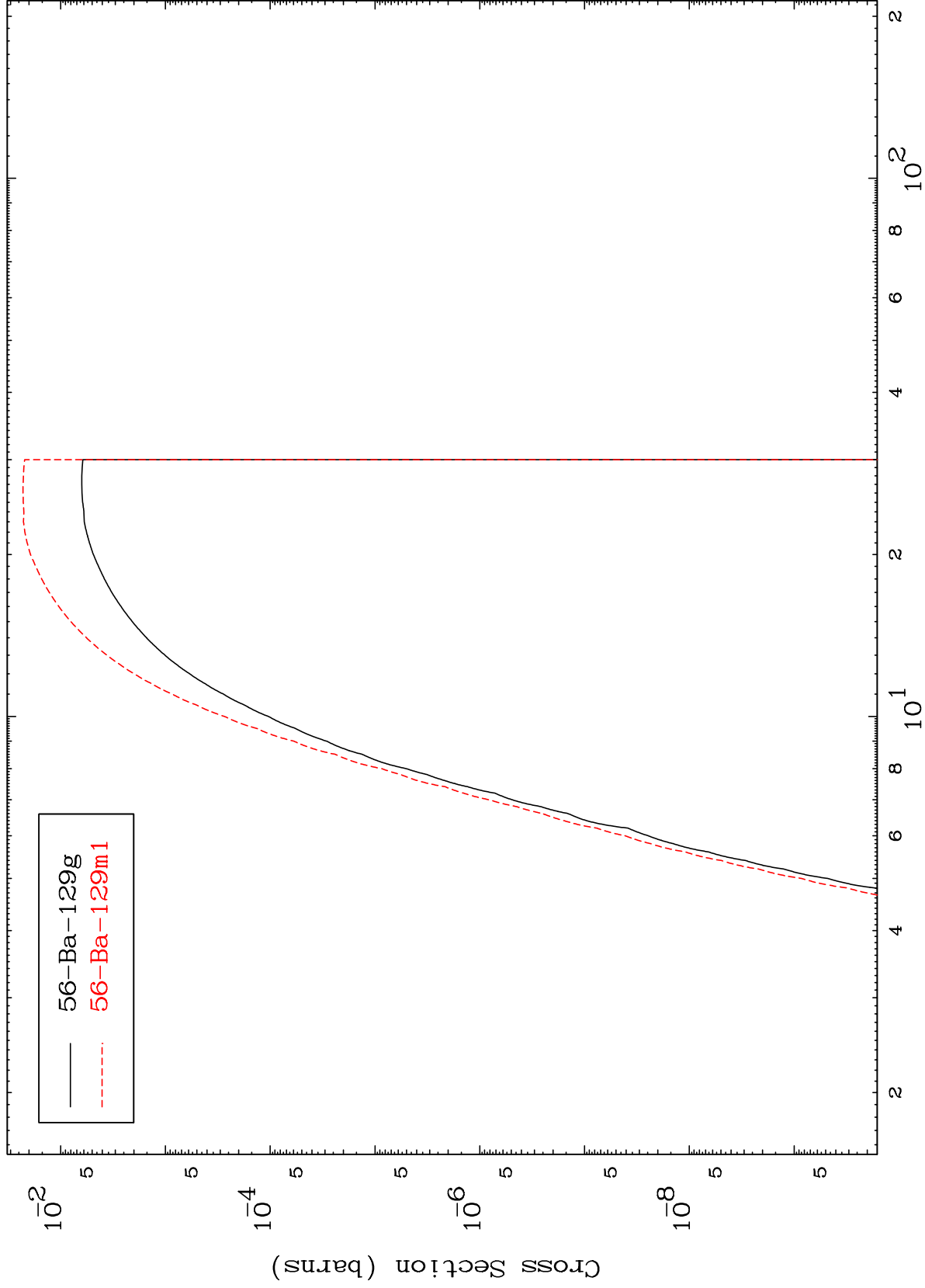
Incident Energy (MeV)

57-La-130

MAT 5701

57-La-130

(n,d)  
Radionuclide Production Cross Section



79

Incident Energy (MeV)

57-La-130

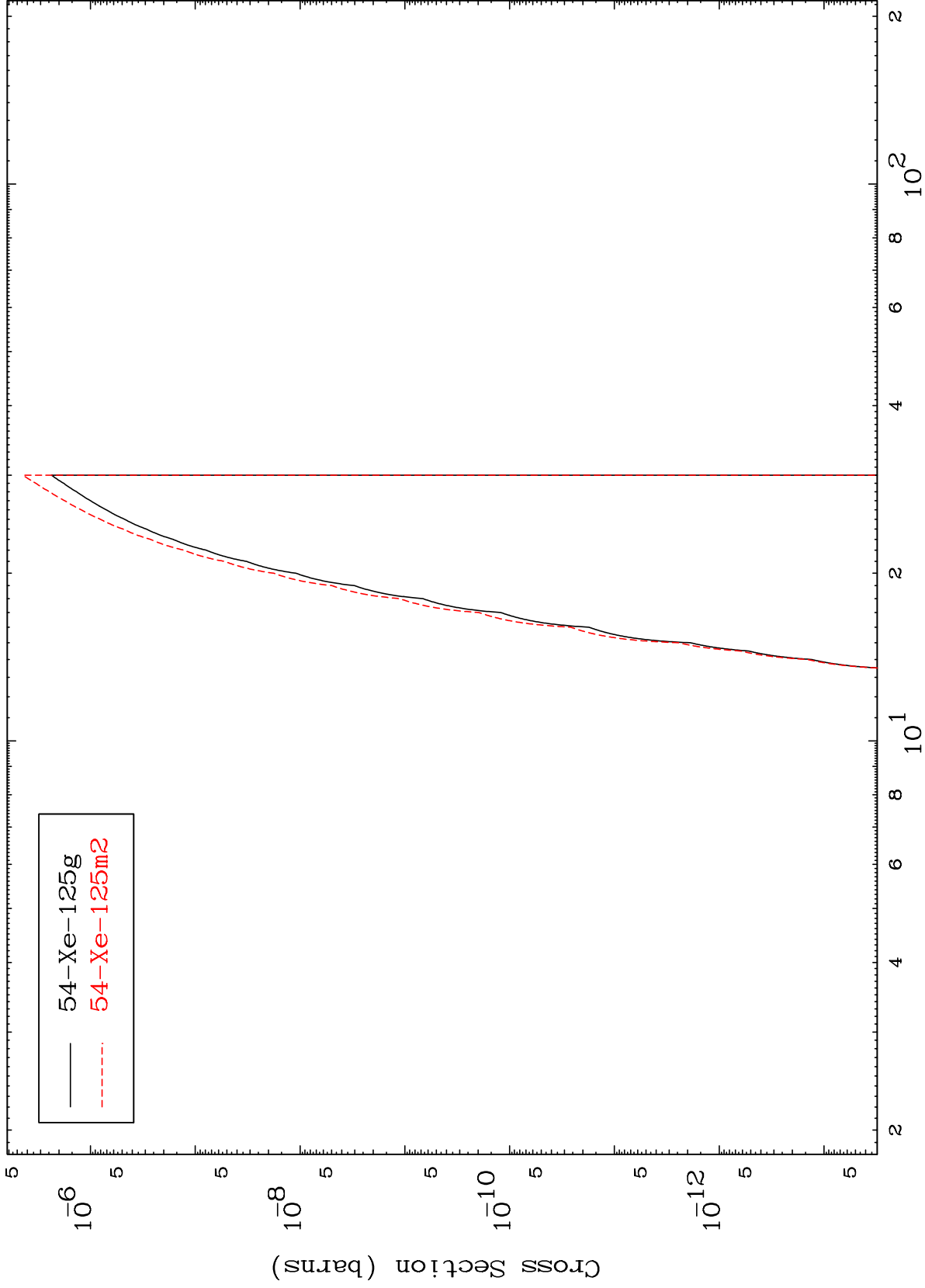


MAT 5701

(n,d)  $\alpha$

57-La-130

Radionuclide Production Cross Section



80

Incident Energy (MeV)

57-La-130