

Program EVALPLOT
(Version 2021-1)

by

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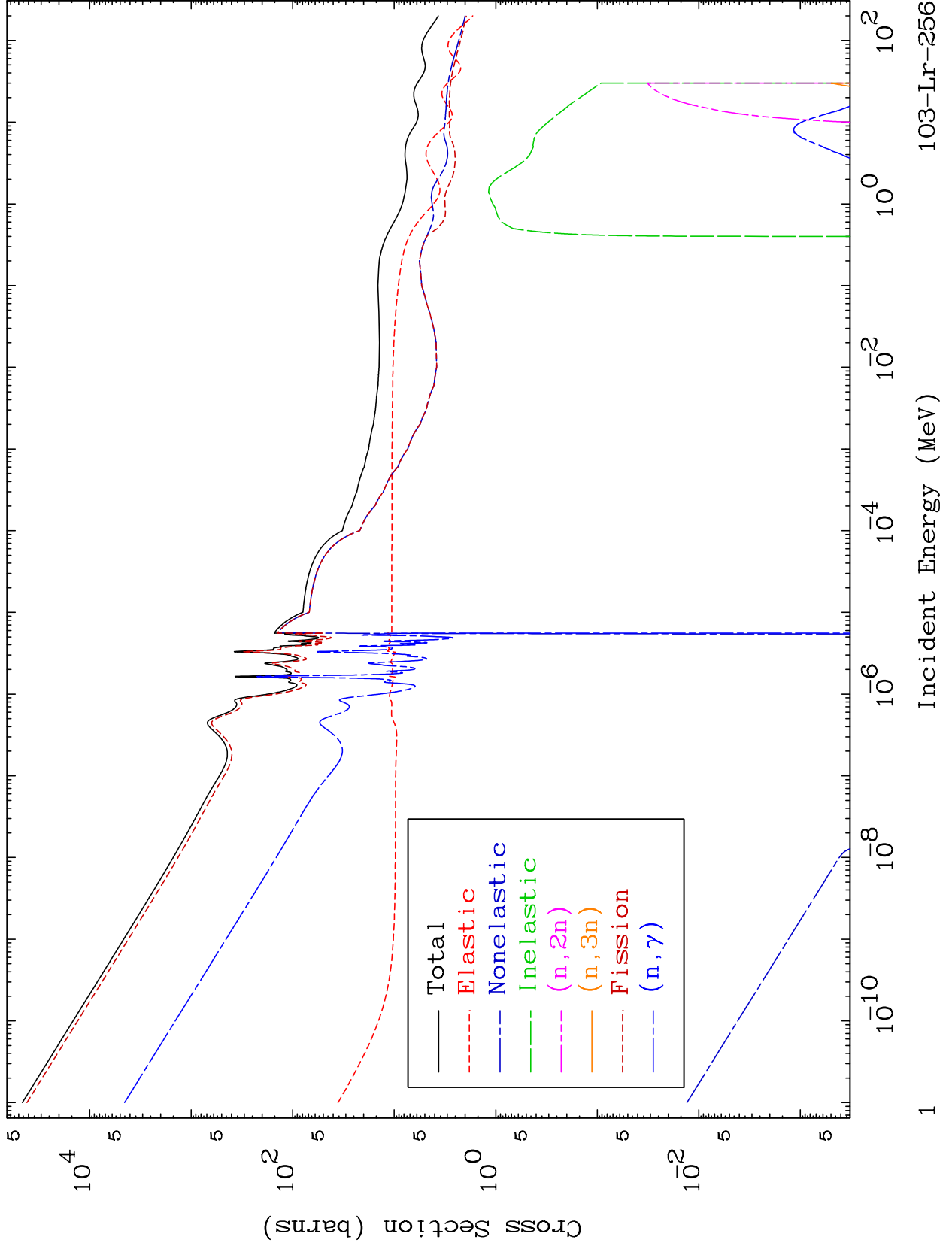
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9985

Neutron Major
293 Kelvin Cross Sections

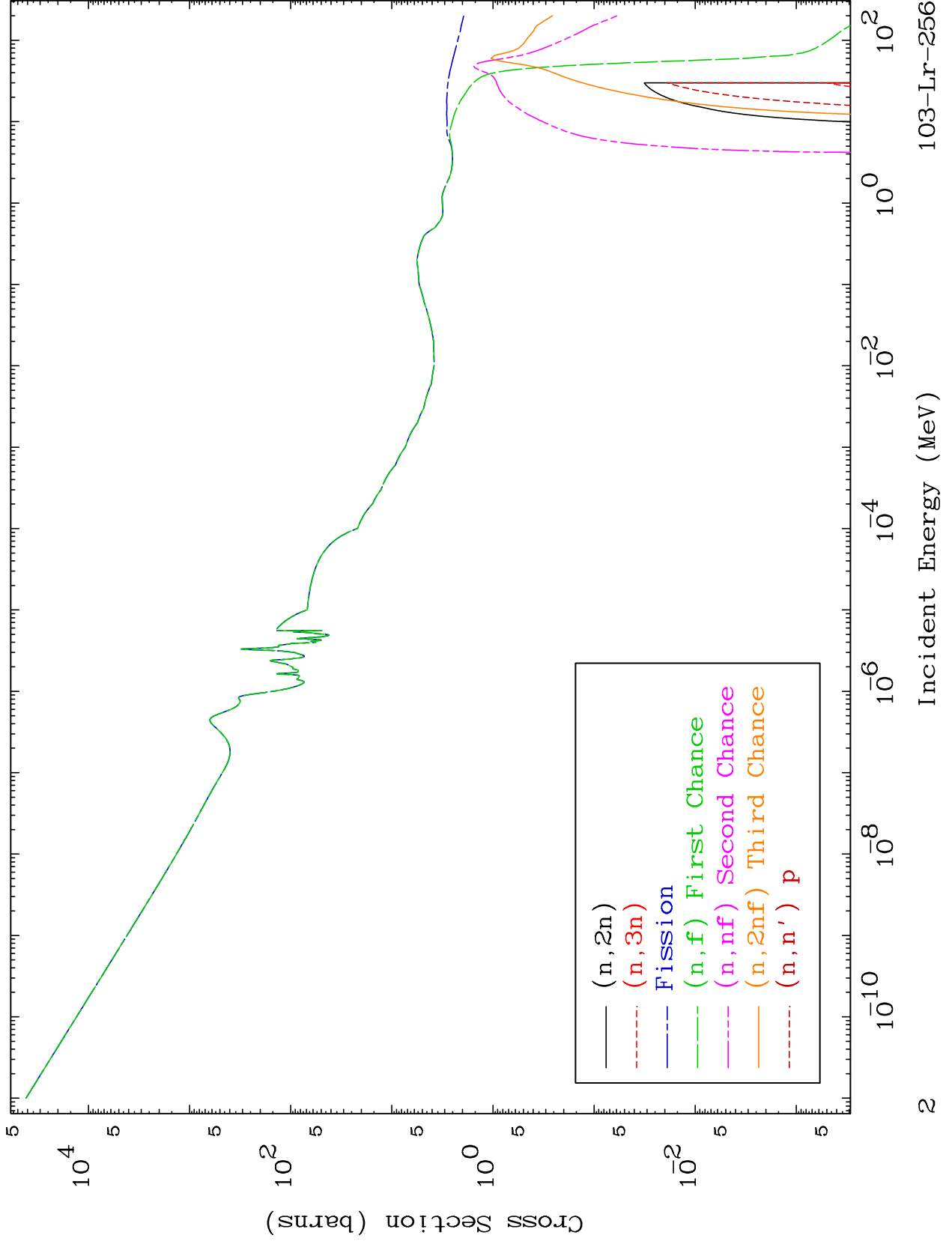
103-Lr-256

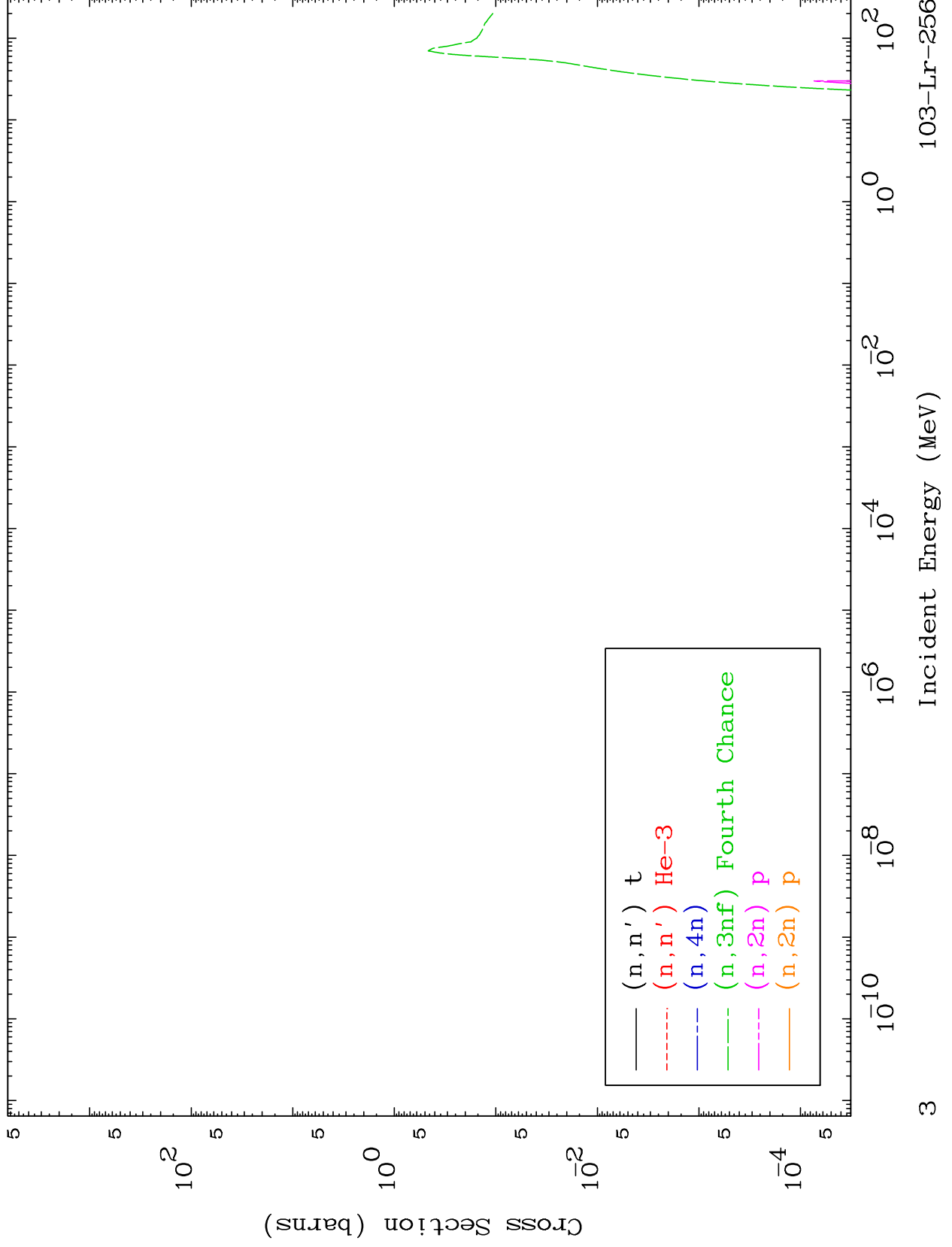


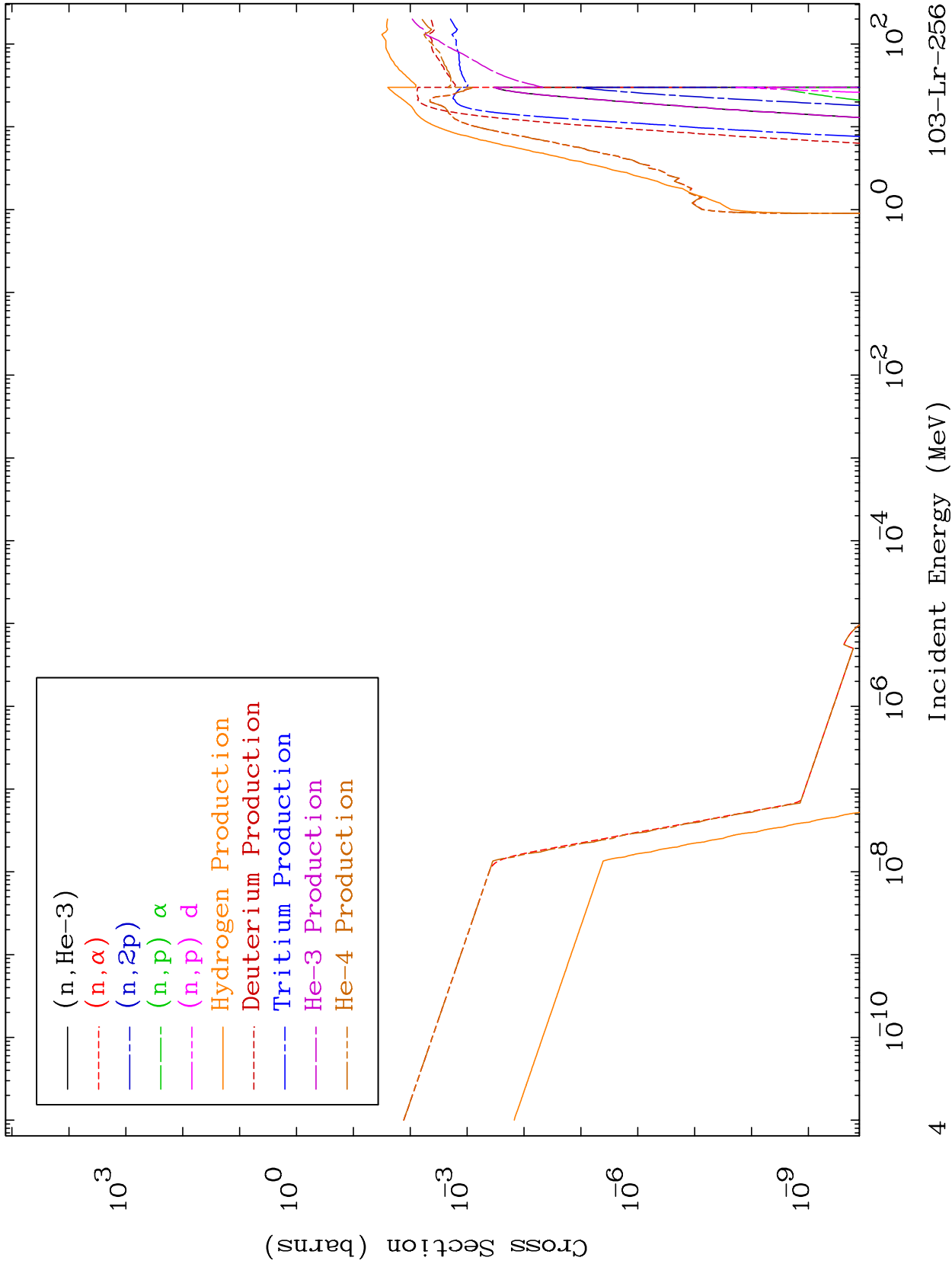
MAT 9985

Neutron Absorption
293 Kelvin Cross Sections

103-Lr-256



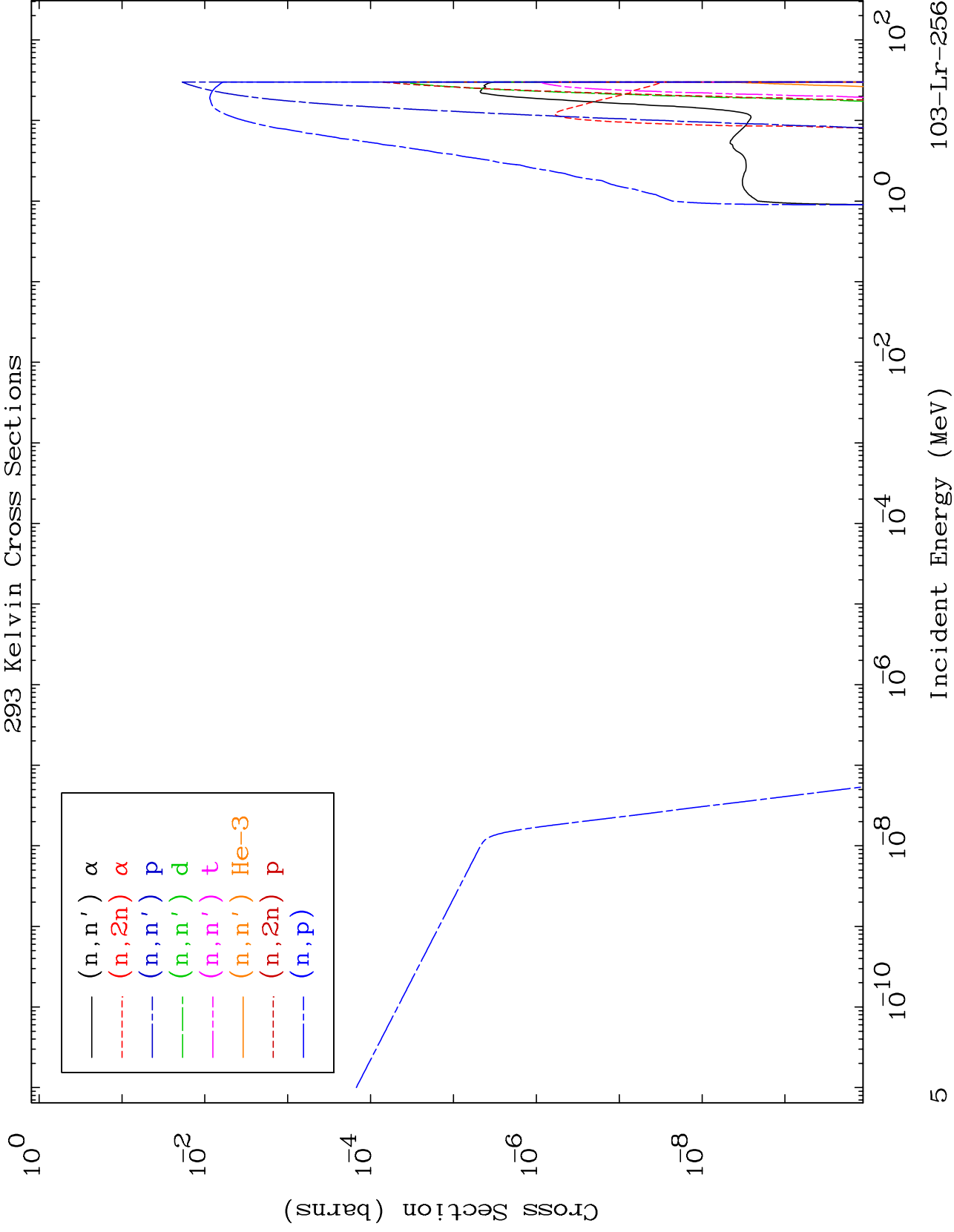




MAT 9985

Charged Particle
293 Kelvin Cross Sections

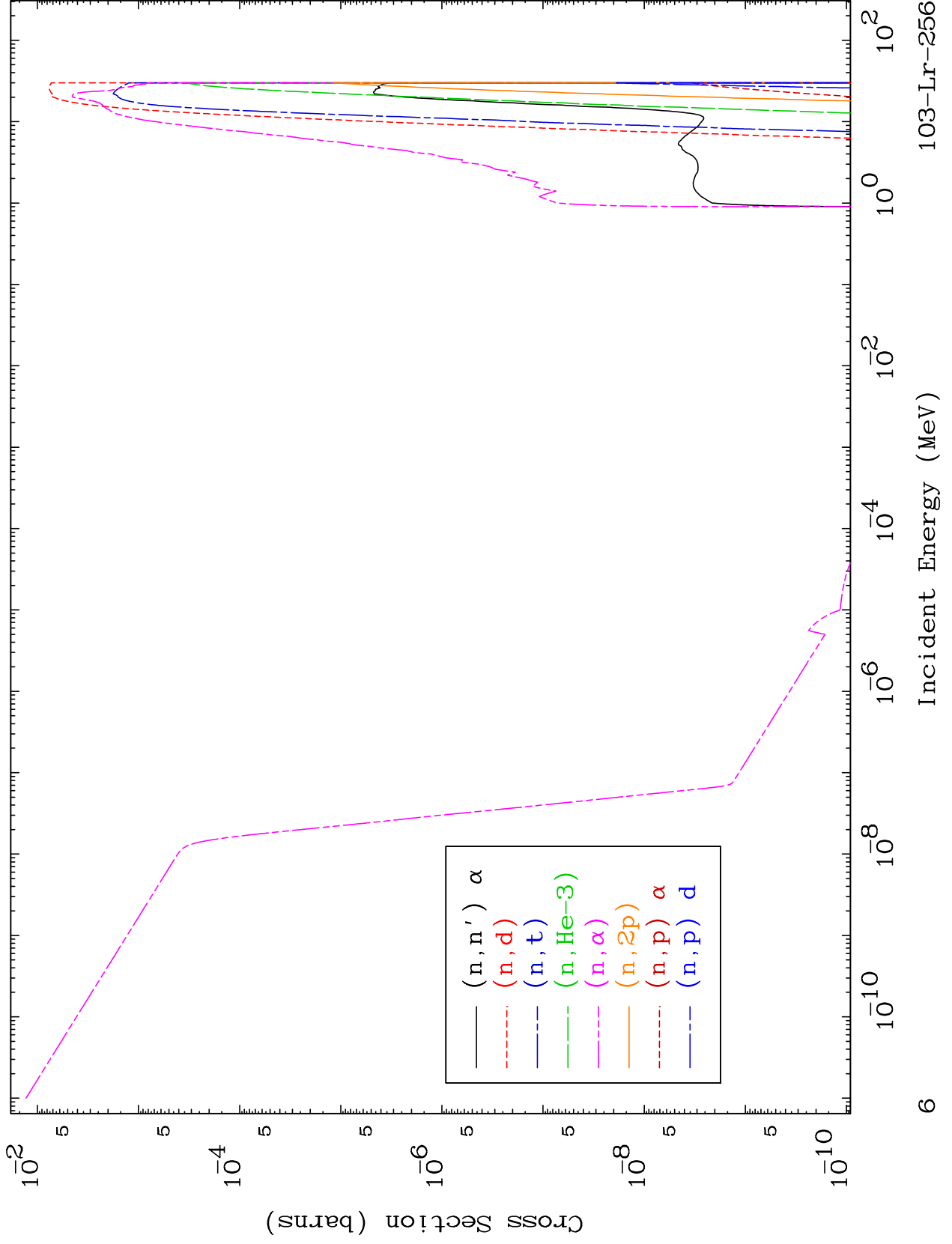
103-Lr-256



MAT 9985

Charged Particle
293 Kelvin Cross Sections

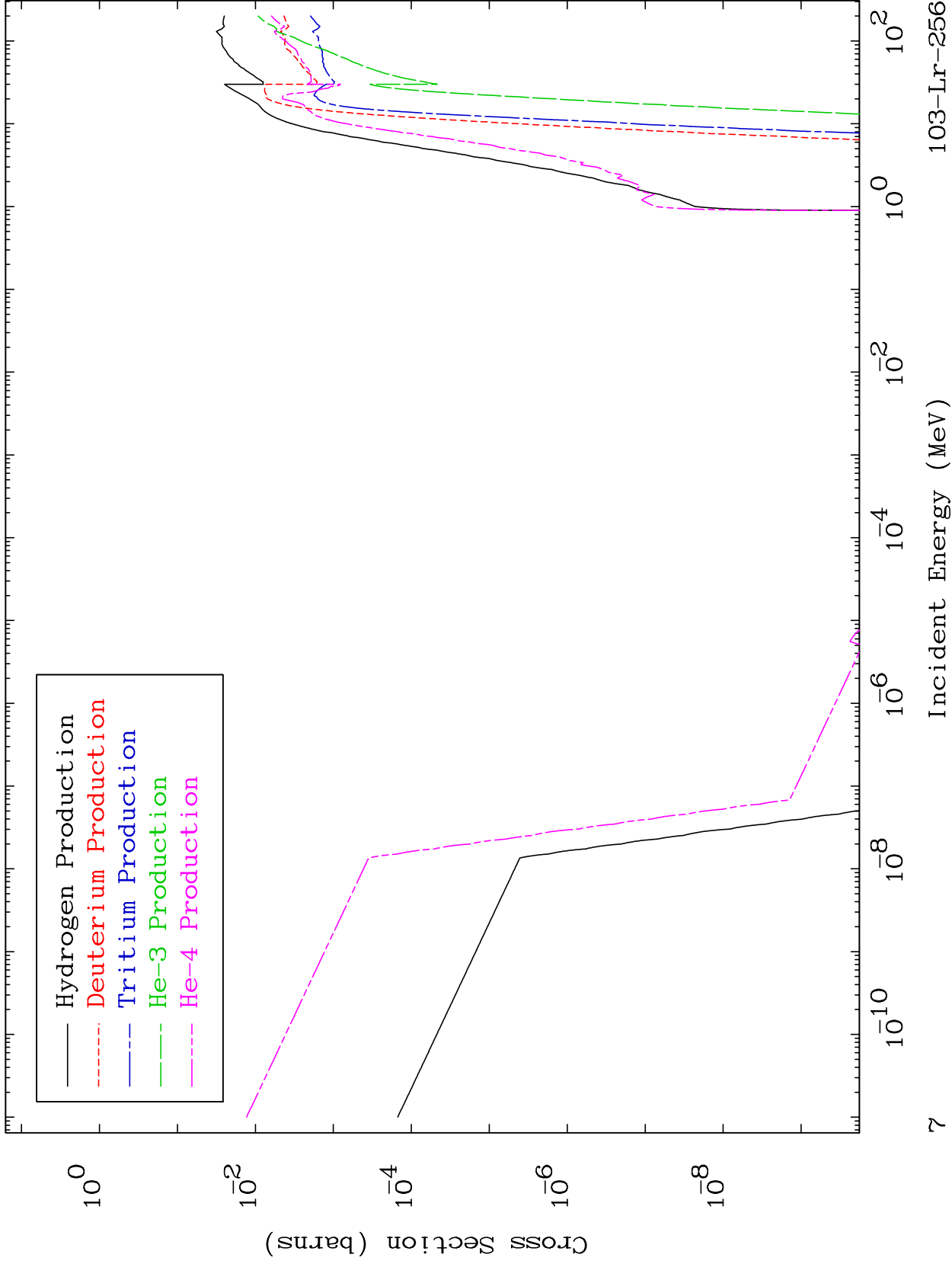
103-Lr-256



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Particle Production
293 Kelvin Cross Sections

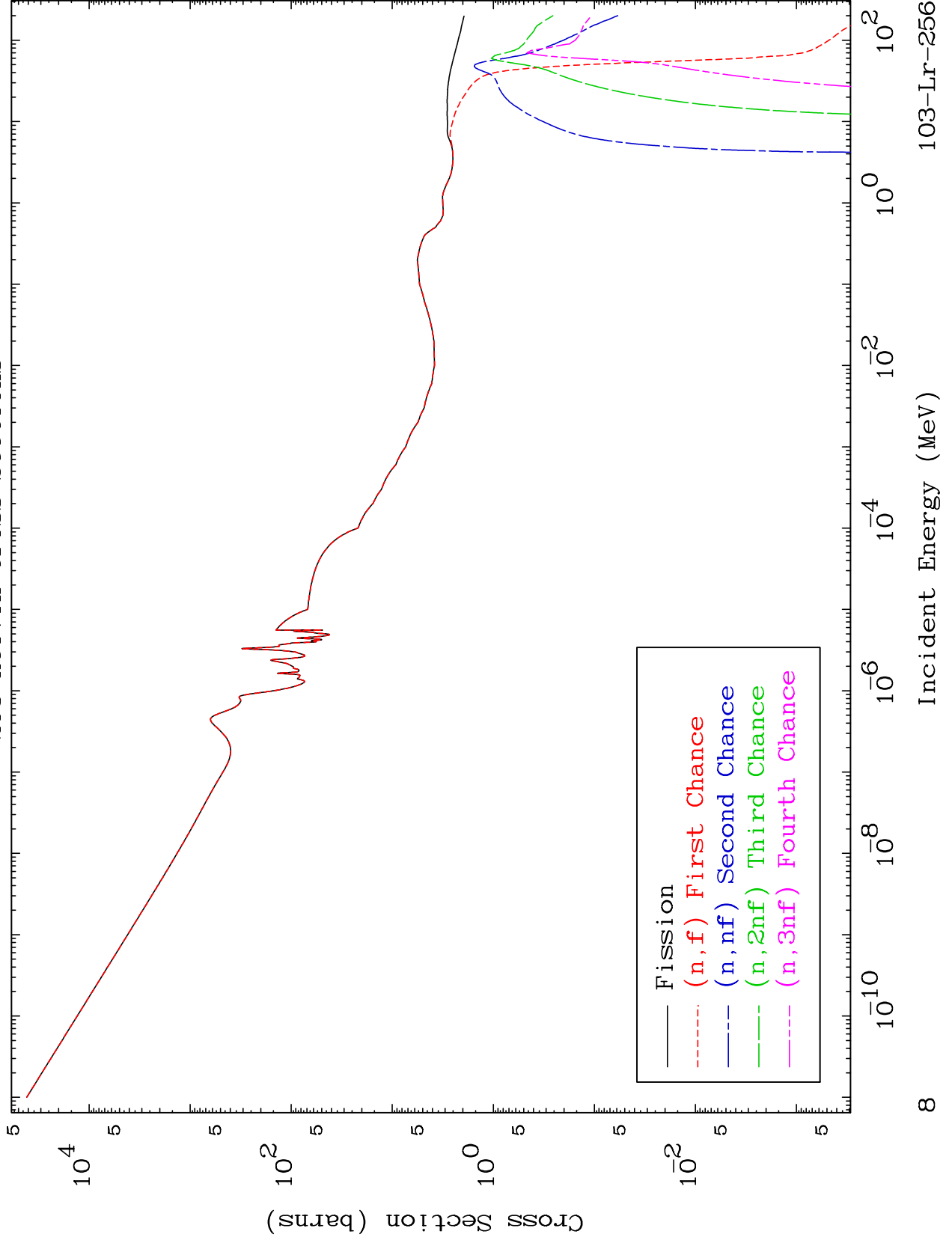
103-Lr-256



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Fission
293 Kelvin Cross Sections

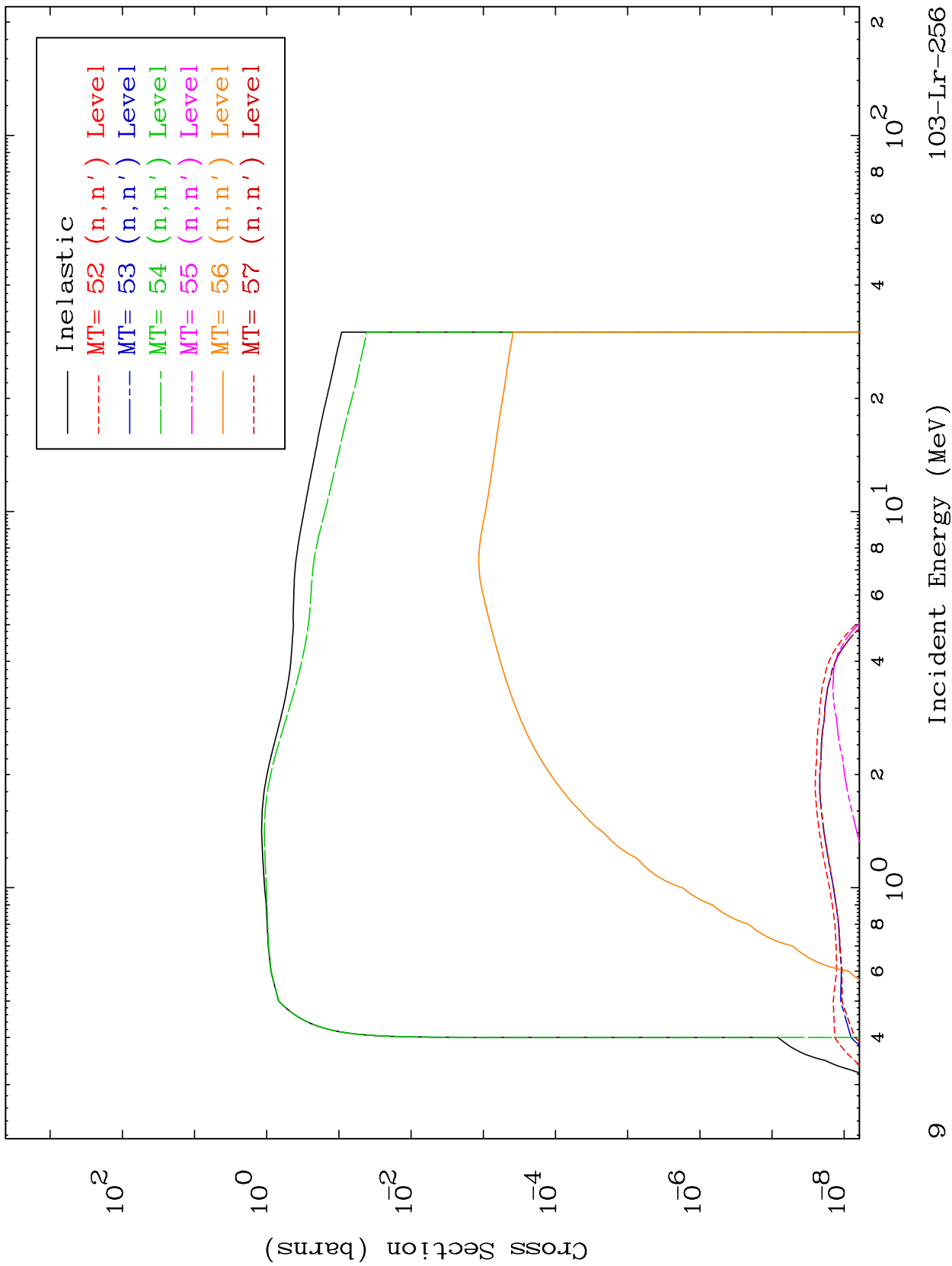
103-Lr-256



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(n,n') Levels
293 Kelvin Cross Sections

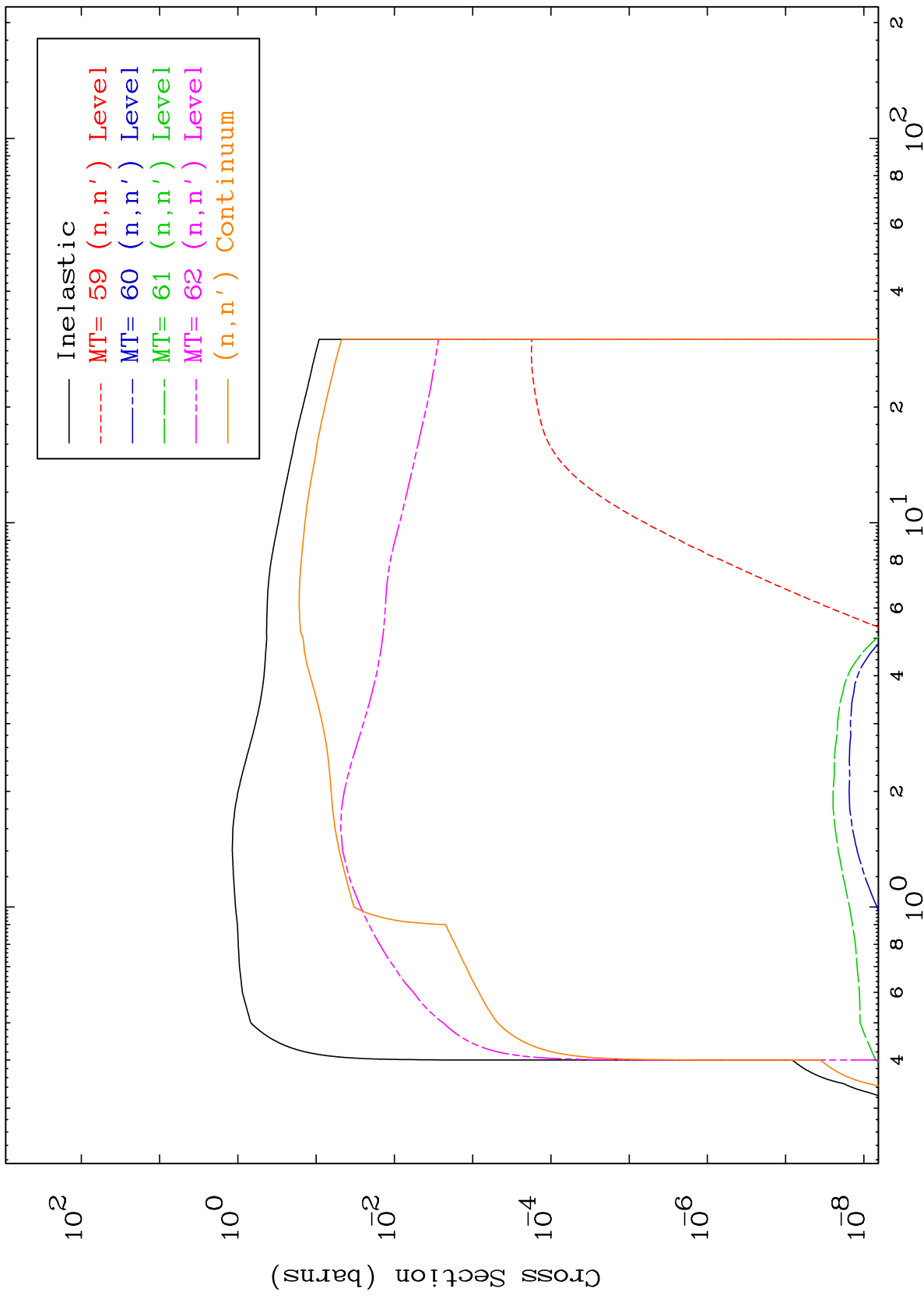
103-Lr-256



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103-Lr-256

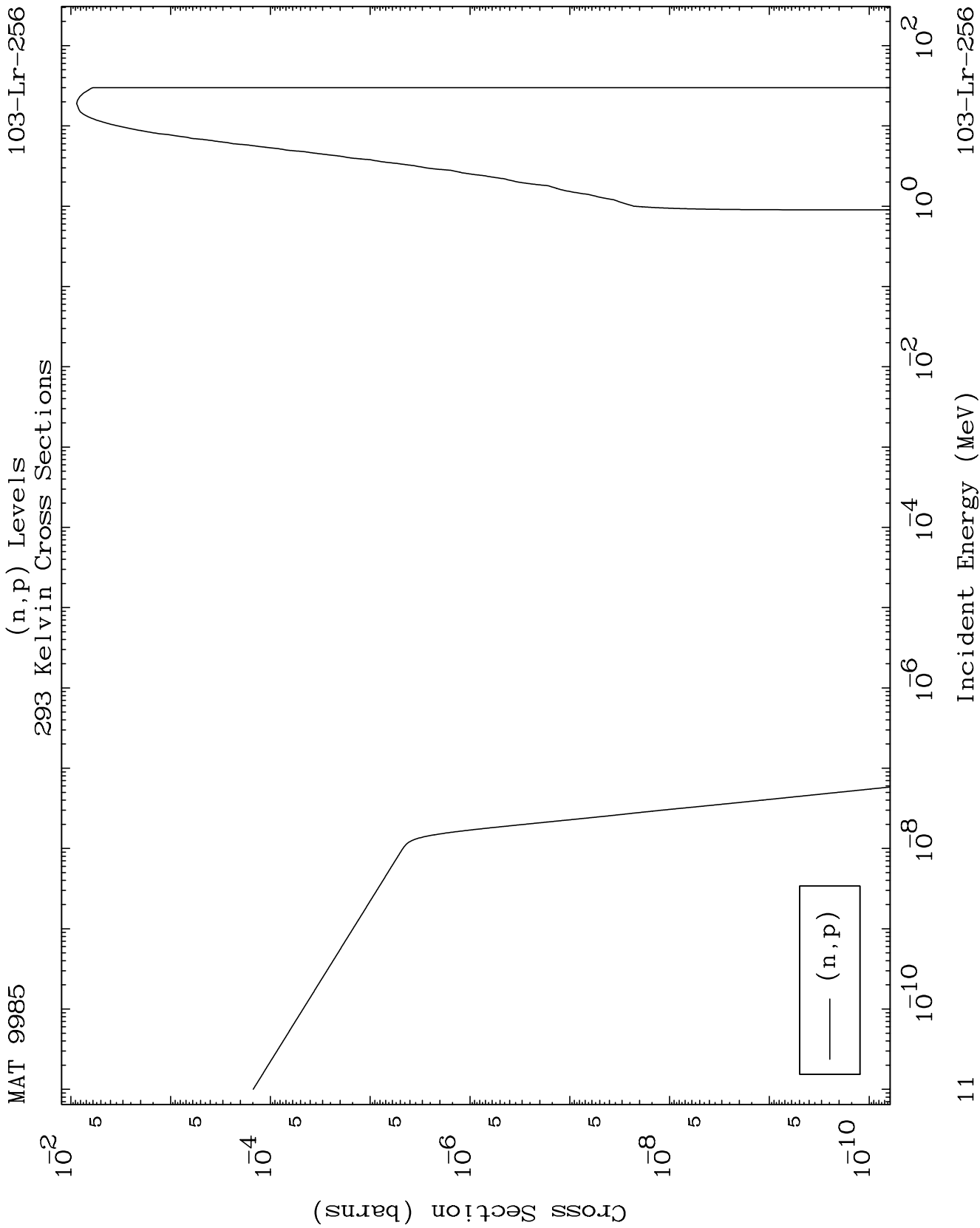
(n,n') Levels
293 Kelvin Cross Sections

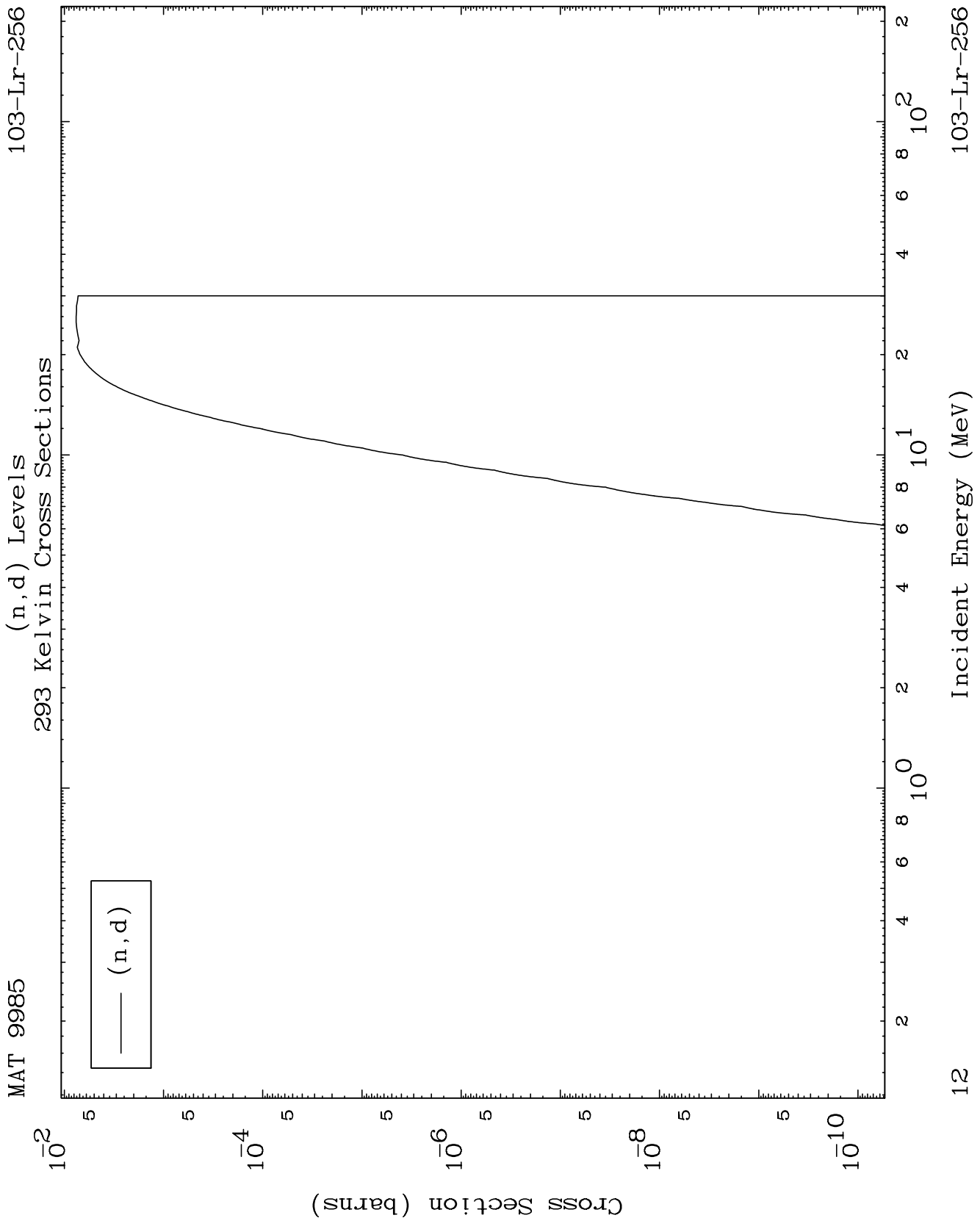


10

Incident Energy (MeV)

103-Lr-256

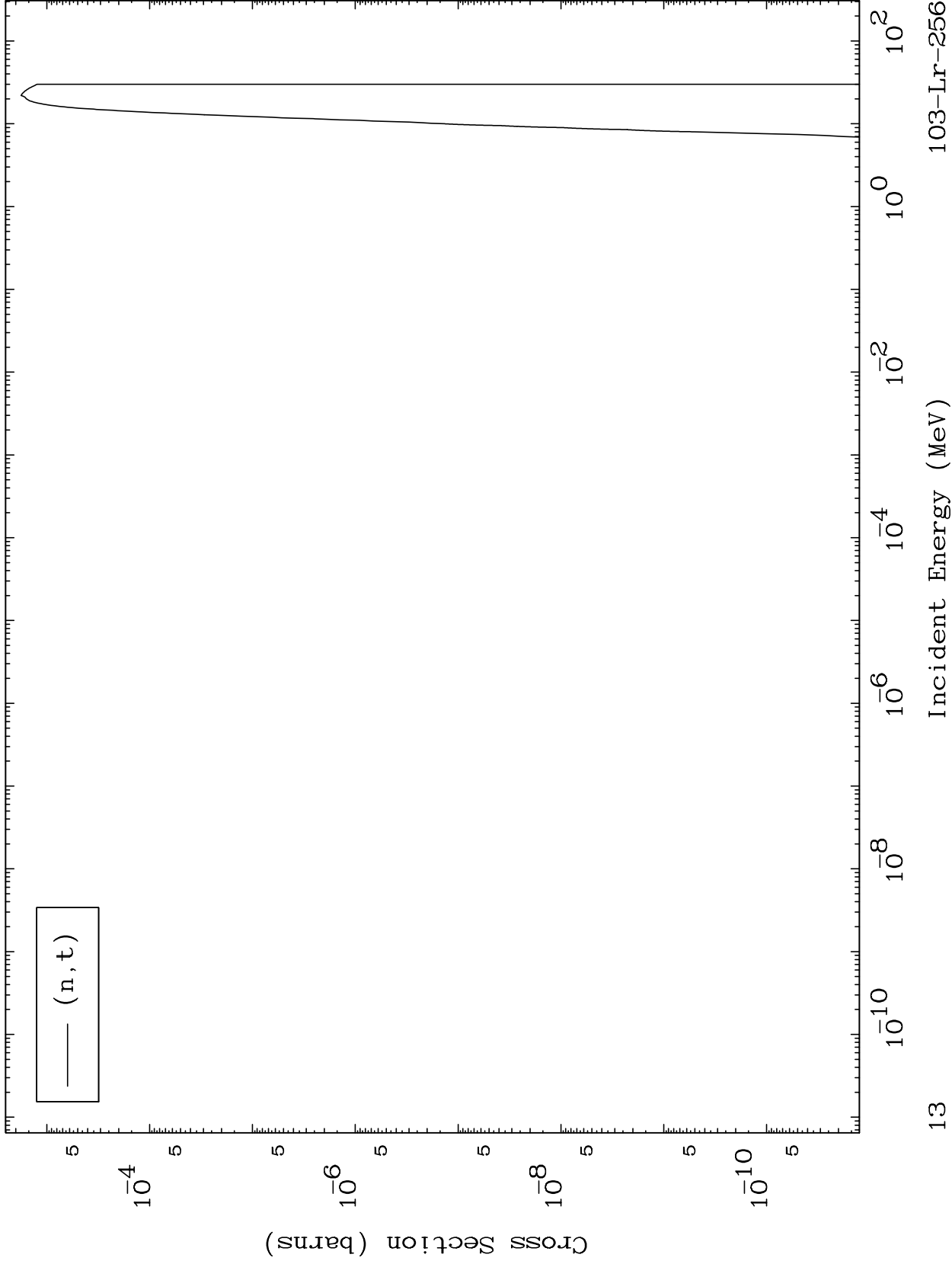




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(n,t) Levels
293 Kelvin Cross Sections

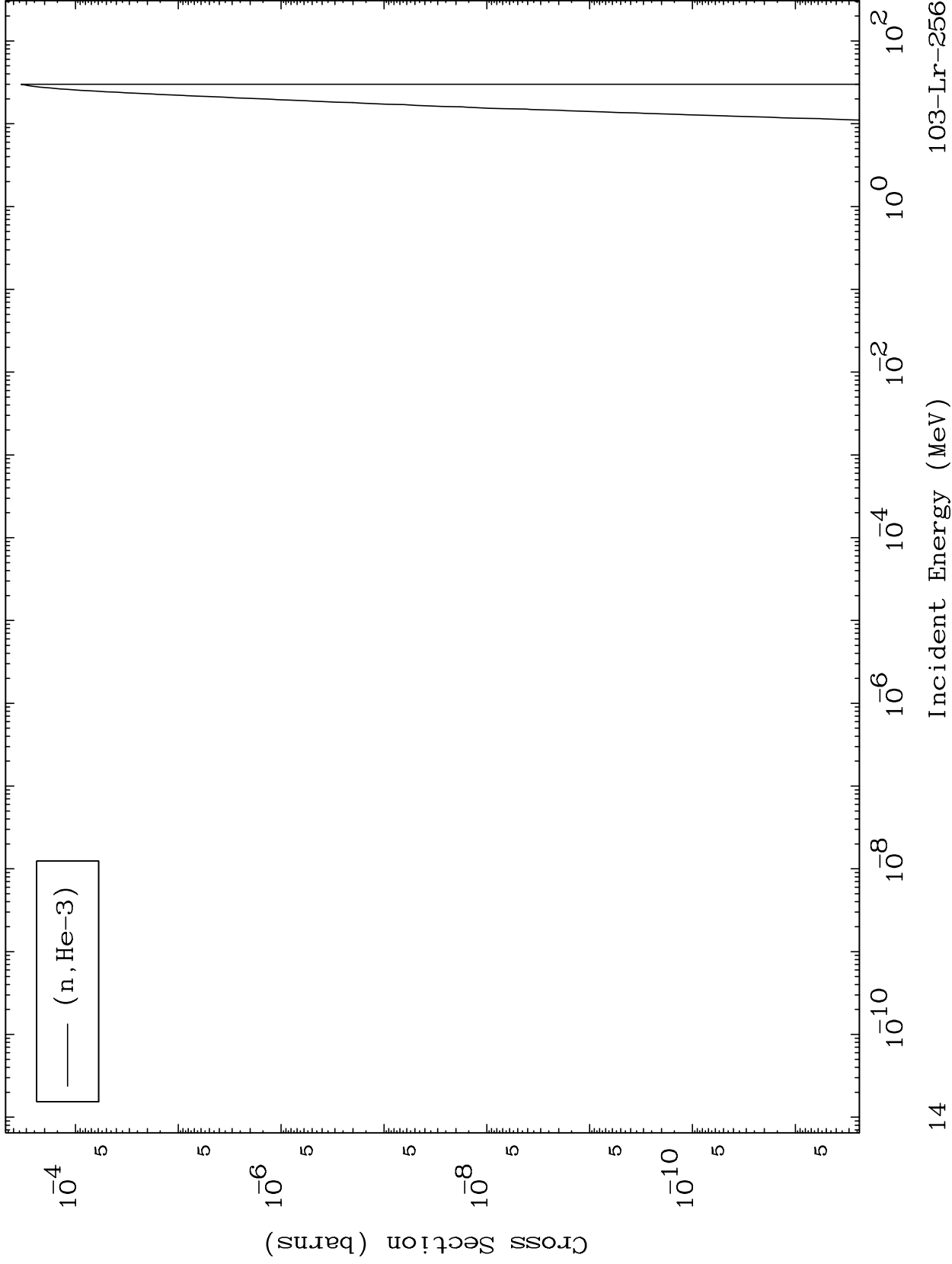
103-Lr-256



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(n,He3) Levels
293 Kelvin Cross Sections

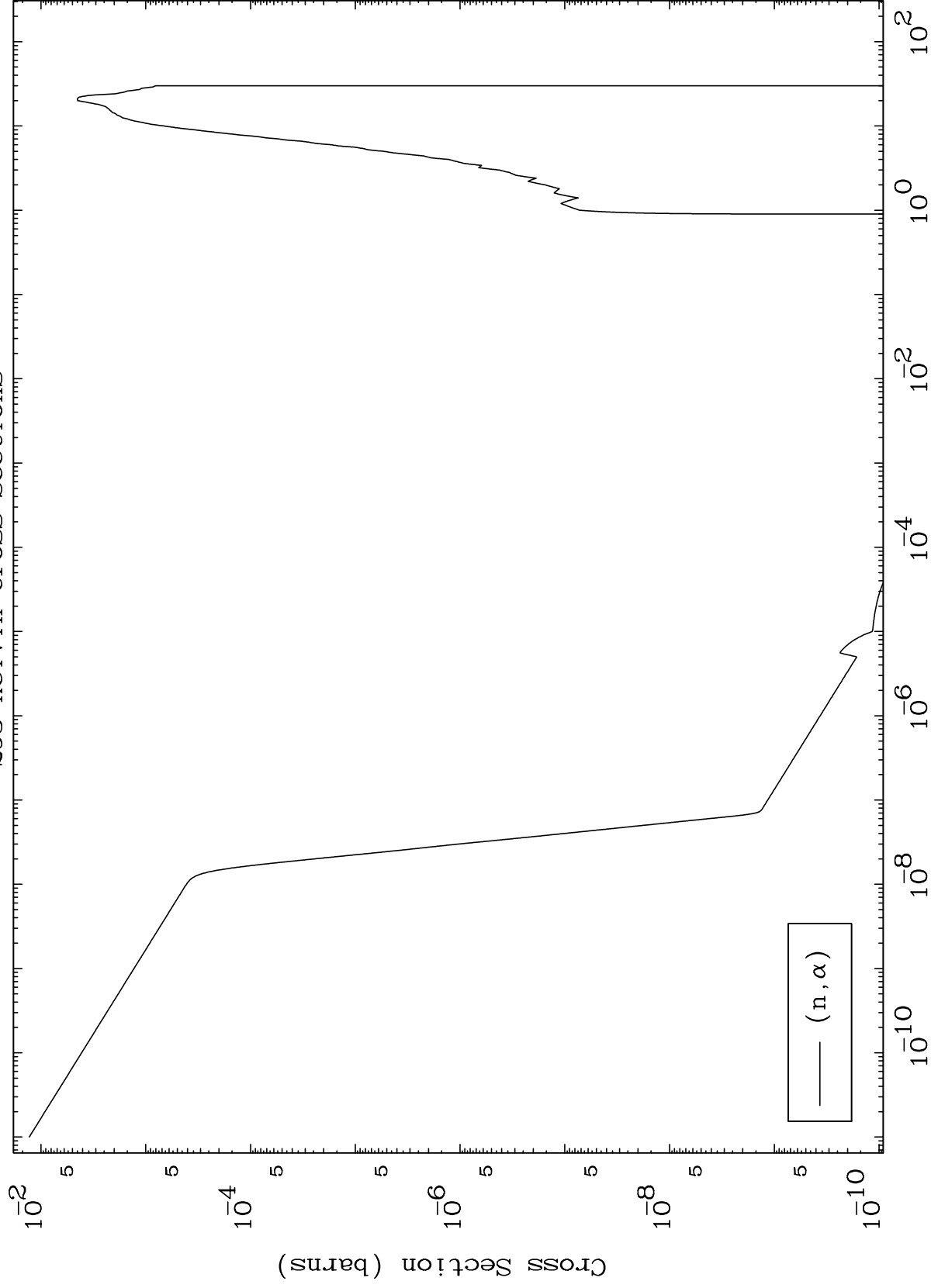
103-Lr-256



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(n, α) Levels
293 Kelvin Cross Sections

103-Lr-256



15

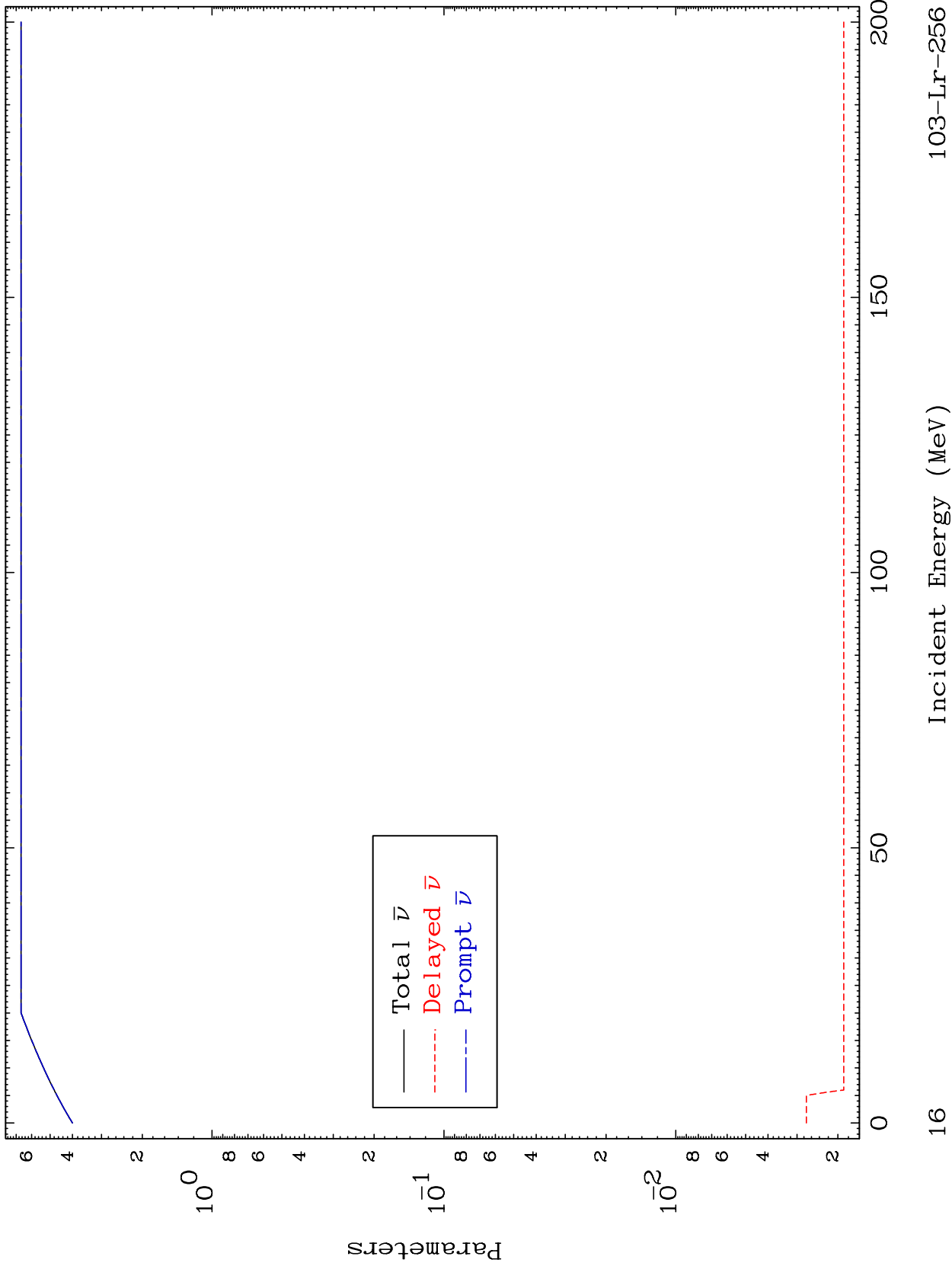
Incident Energy (MeV)

103-Lr-256

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Energy Release
Parameters

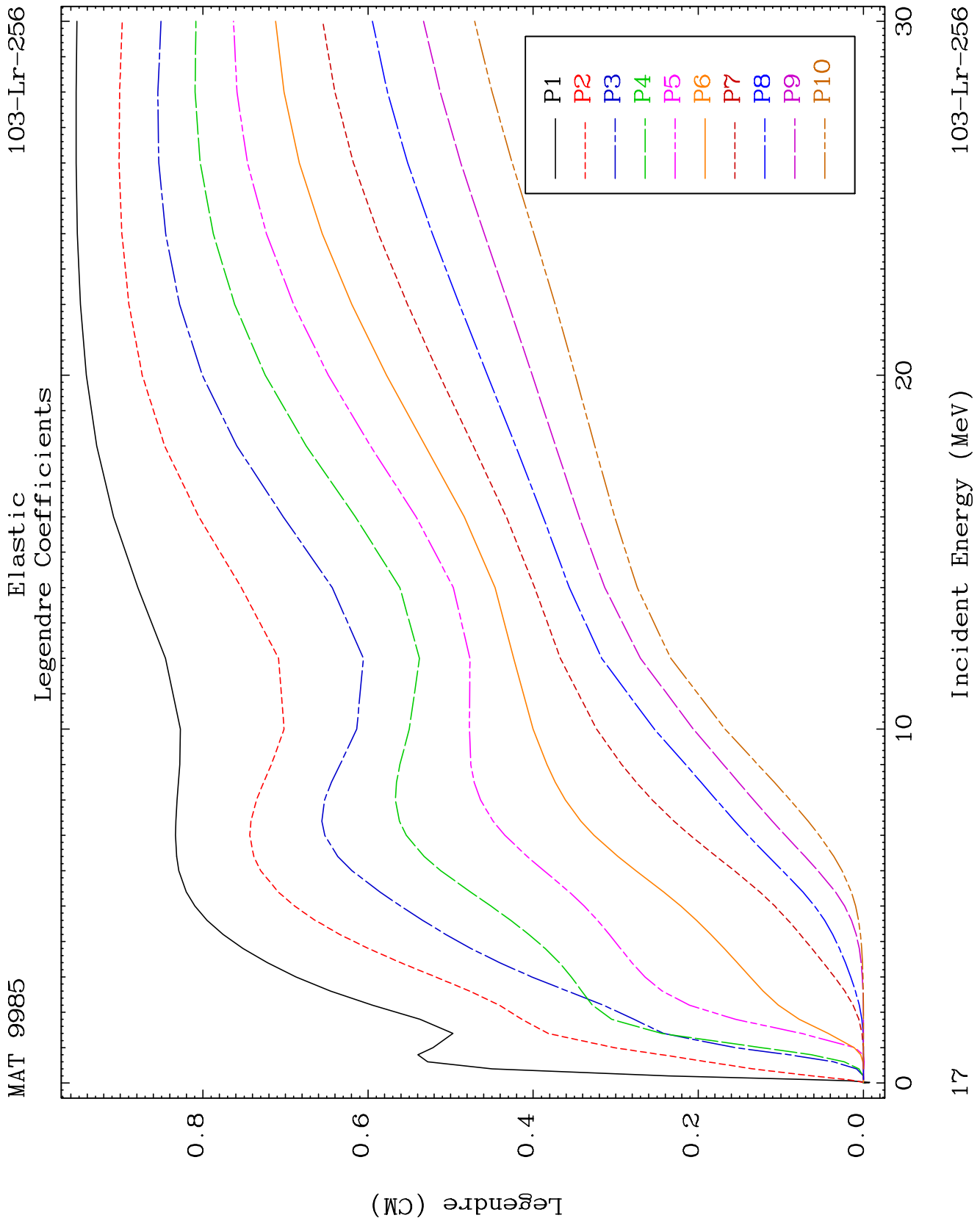
103-Lr-256

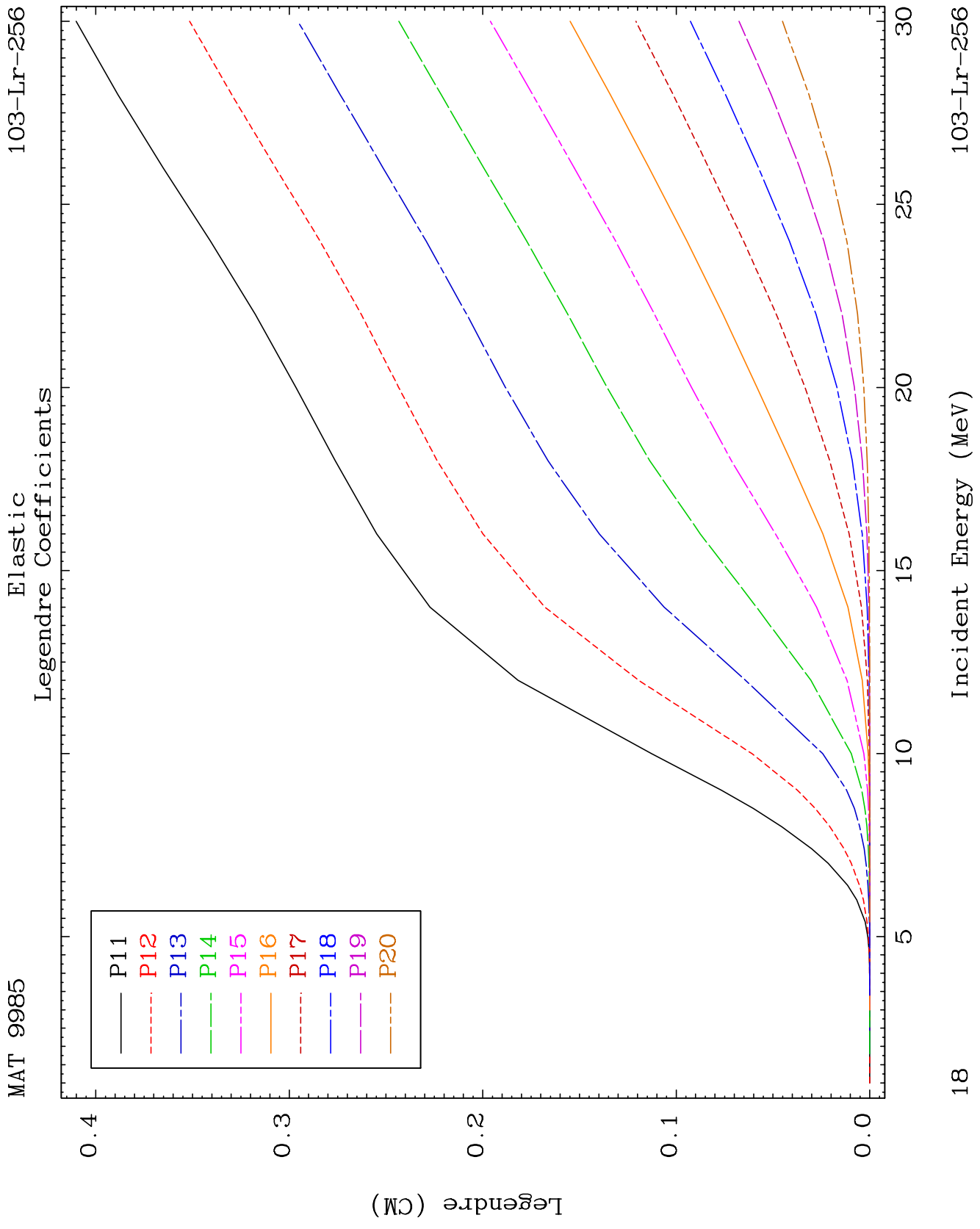


16

Incident Energy (MeV)

103-Lr-256

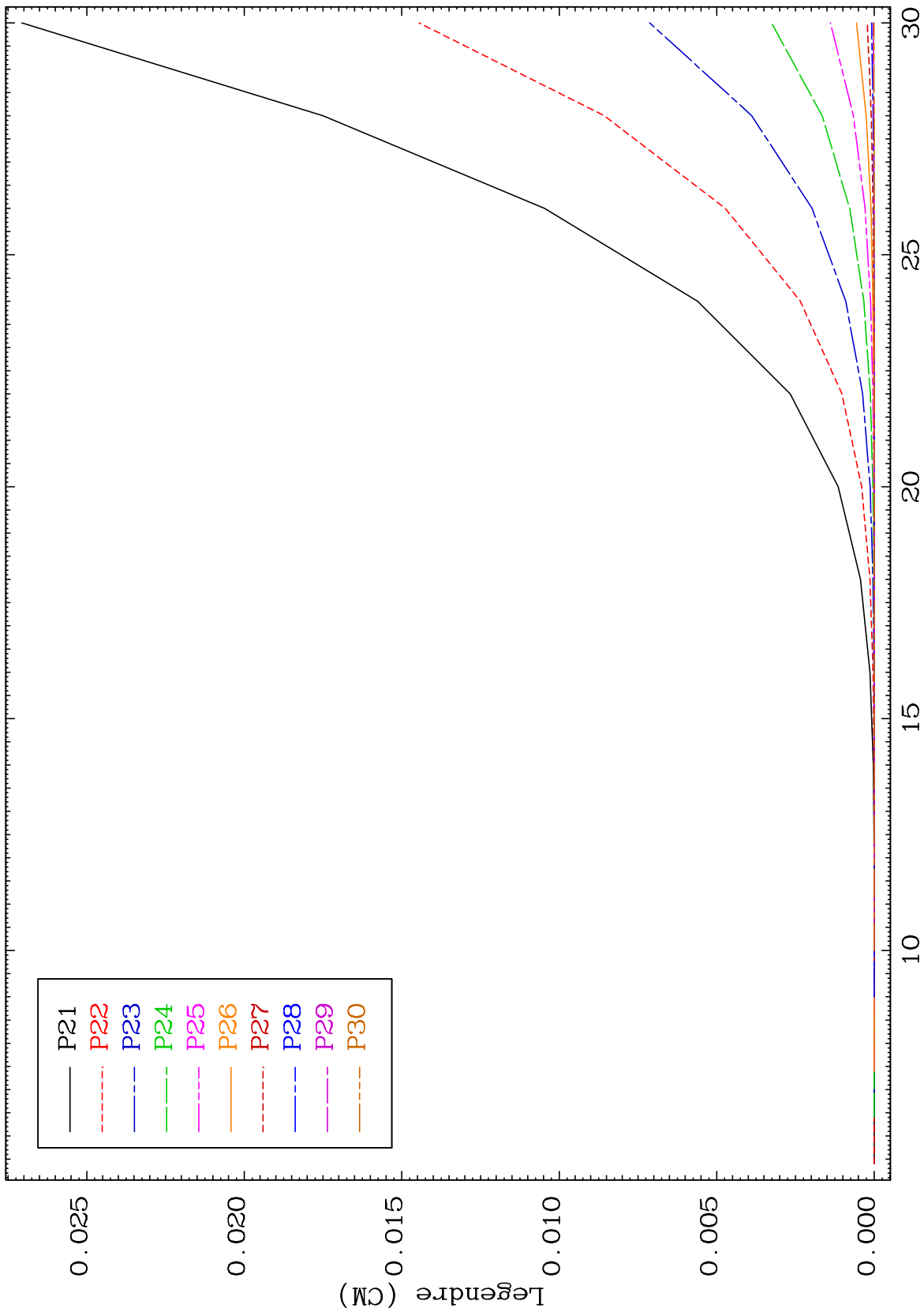




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Elastic Legendre Coefficients

103-Lr-256



19

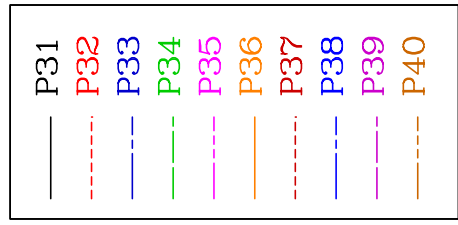
Incident Energy (MeV)

103-Lr-256

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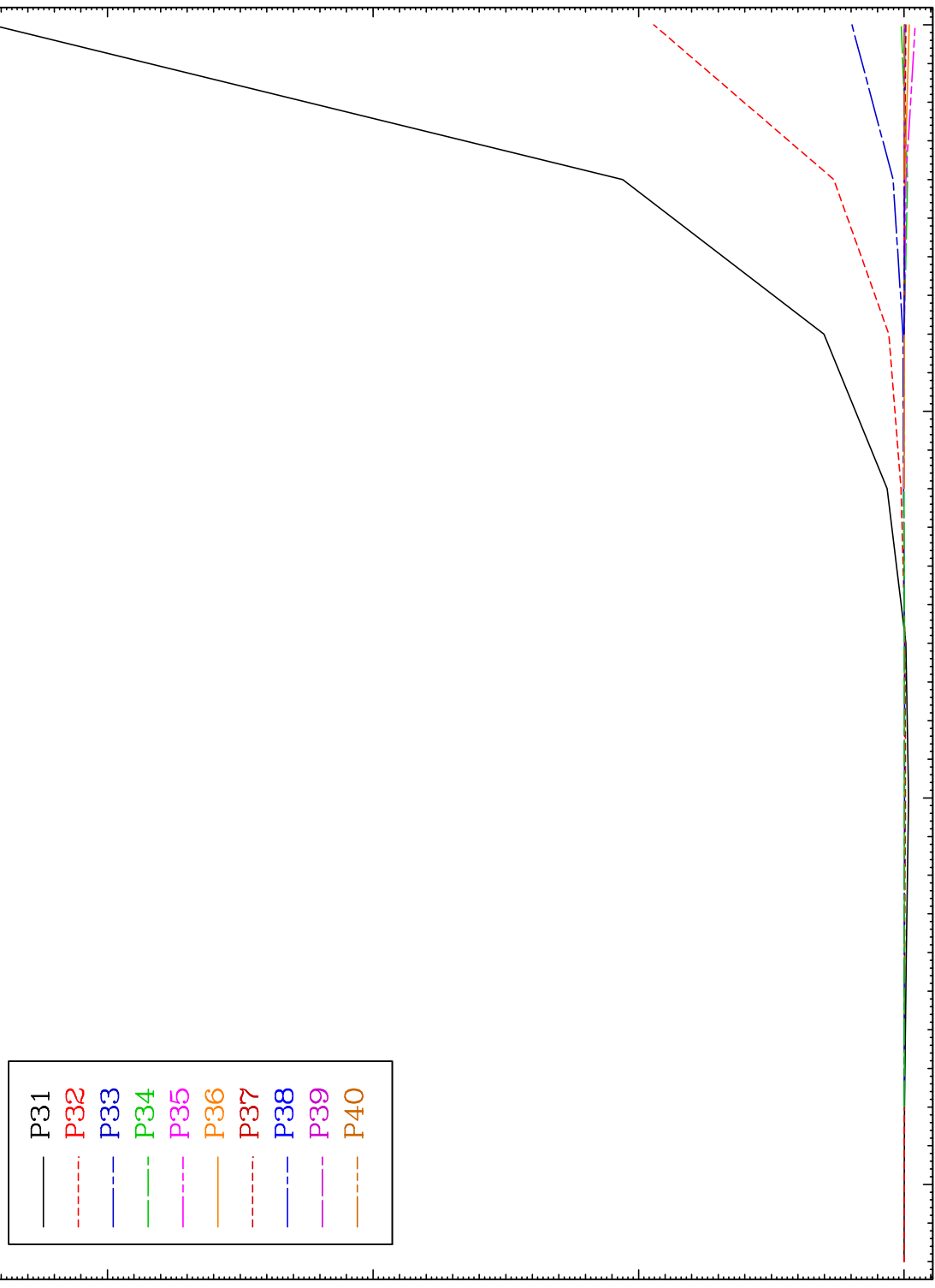
Elastic Legendre Coefficients

103-Lr-256



$\times 10^{-6}$

Legendre (CM)



3

2

1

0

15

20

25

30

20

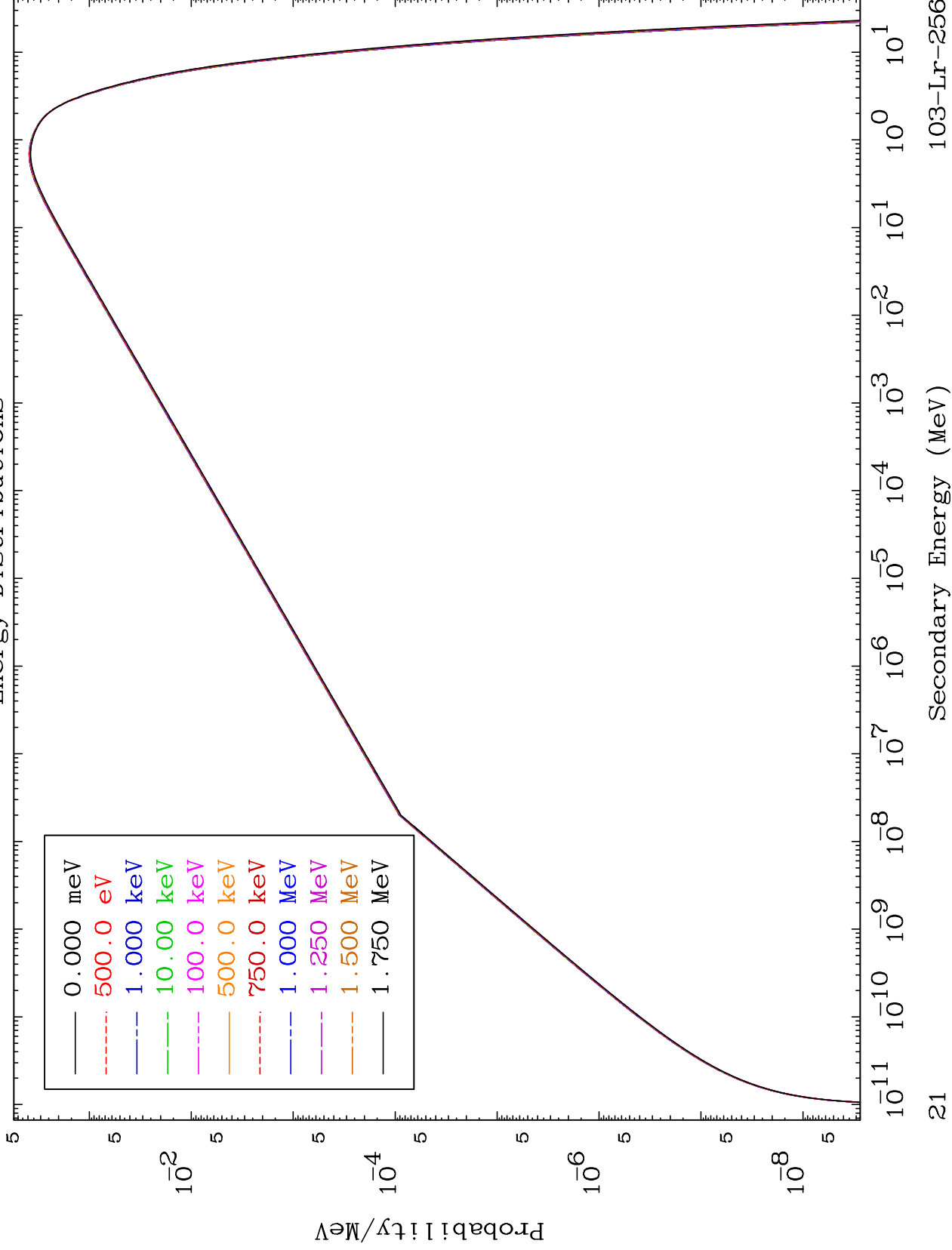
Incident Energy (MeV)

103-Lr-256

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Fission Energy Distributions

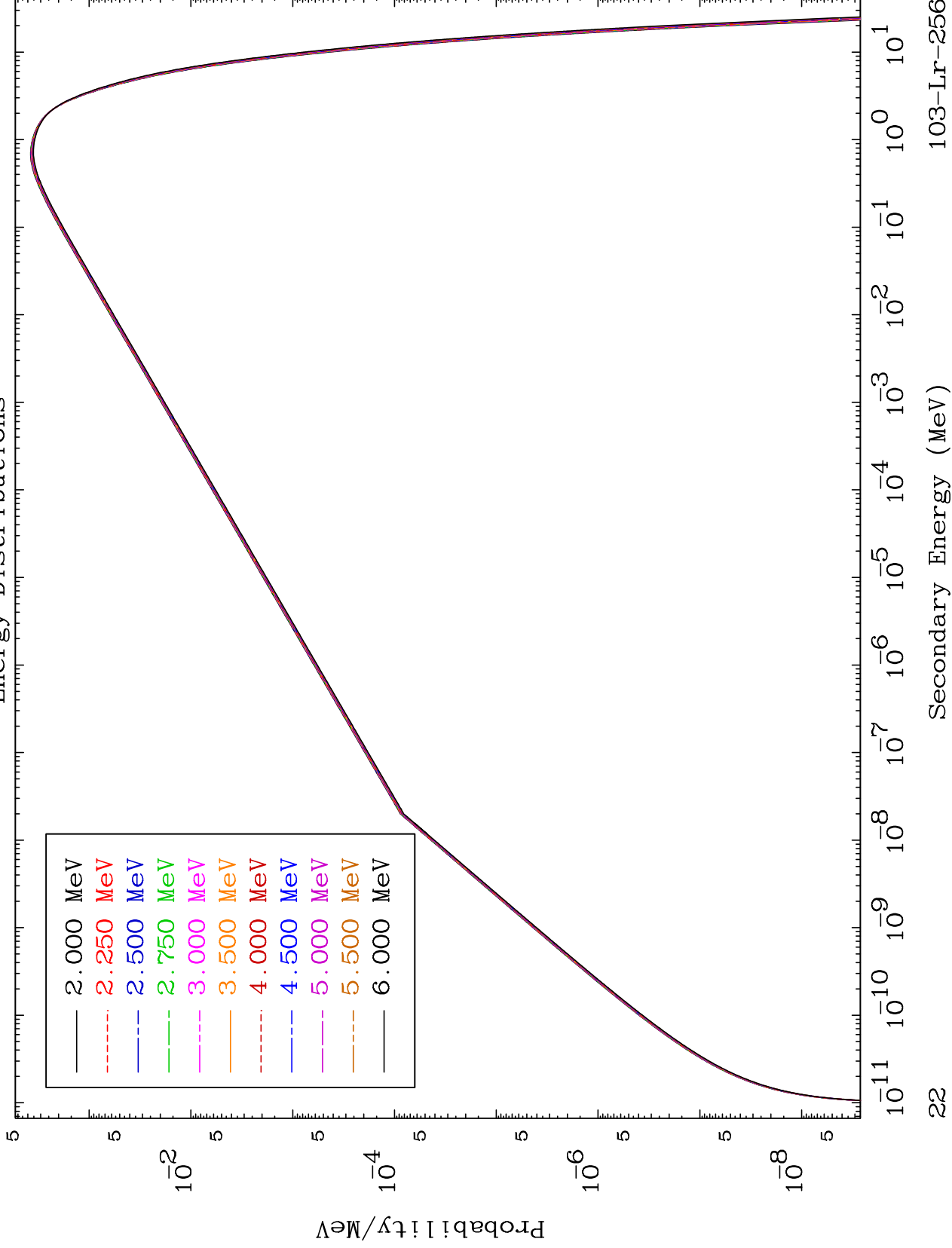
103-Lr-256



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Fission
Energy Distributions

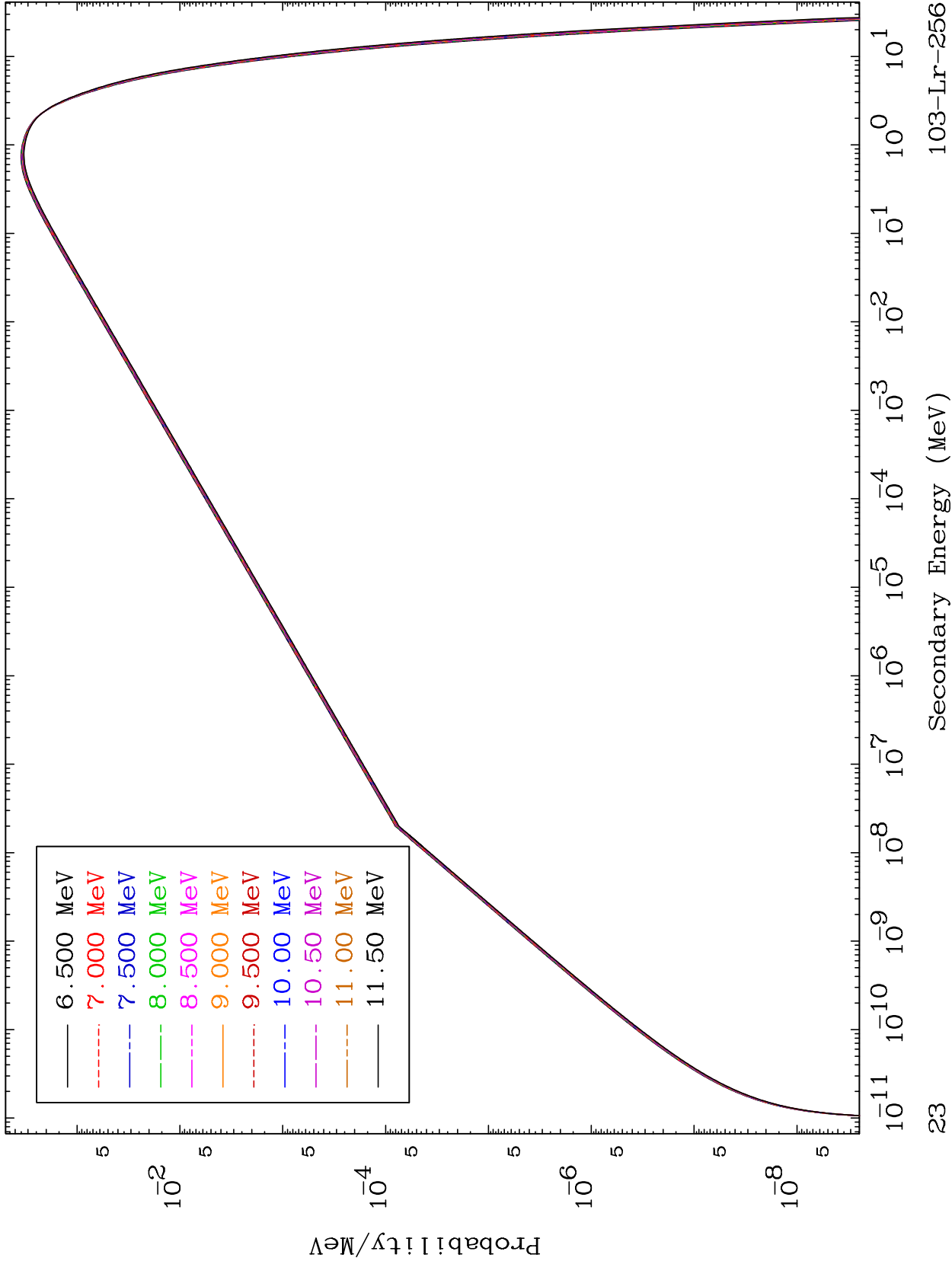
103-Lr-256



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Fission Energy Distributions

103-Lr-256

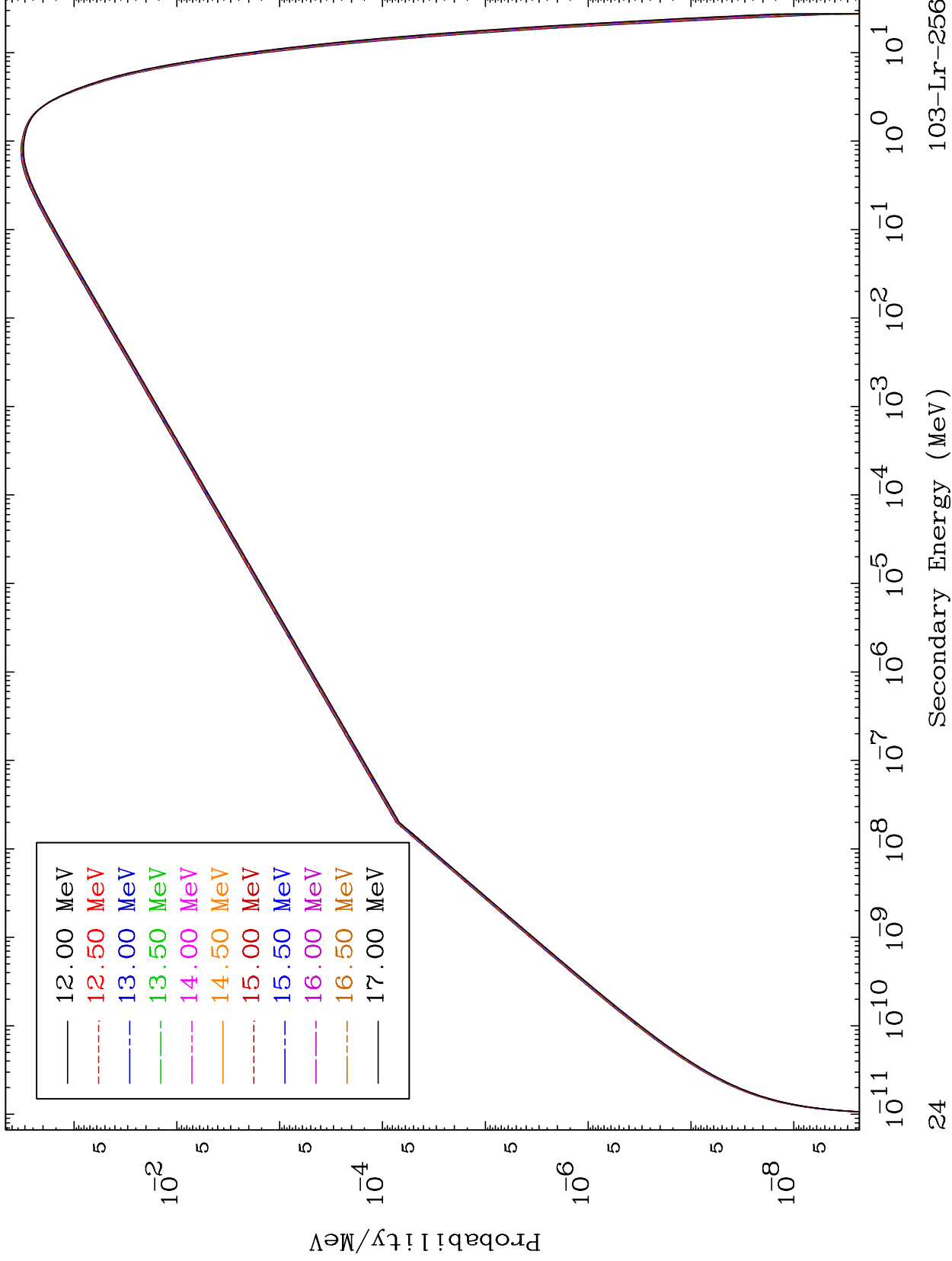


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Fission Energy Distributions

103-Lr-256



103-Lr-256

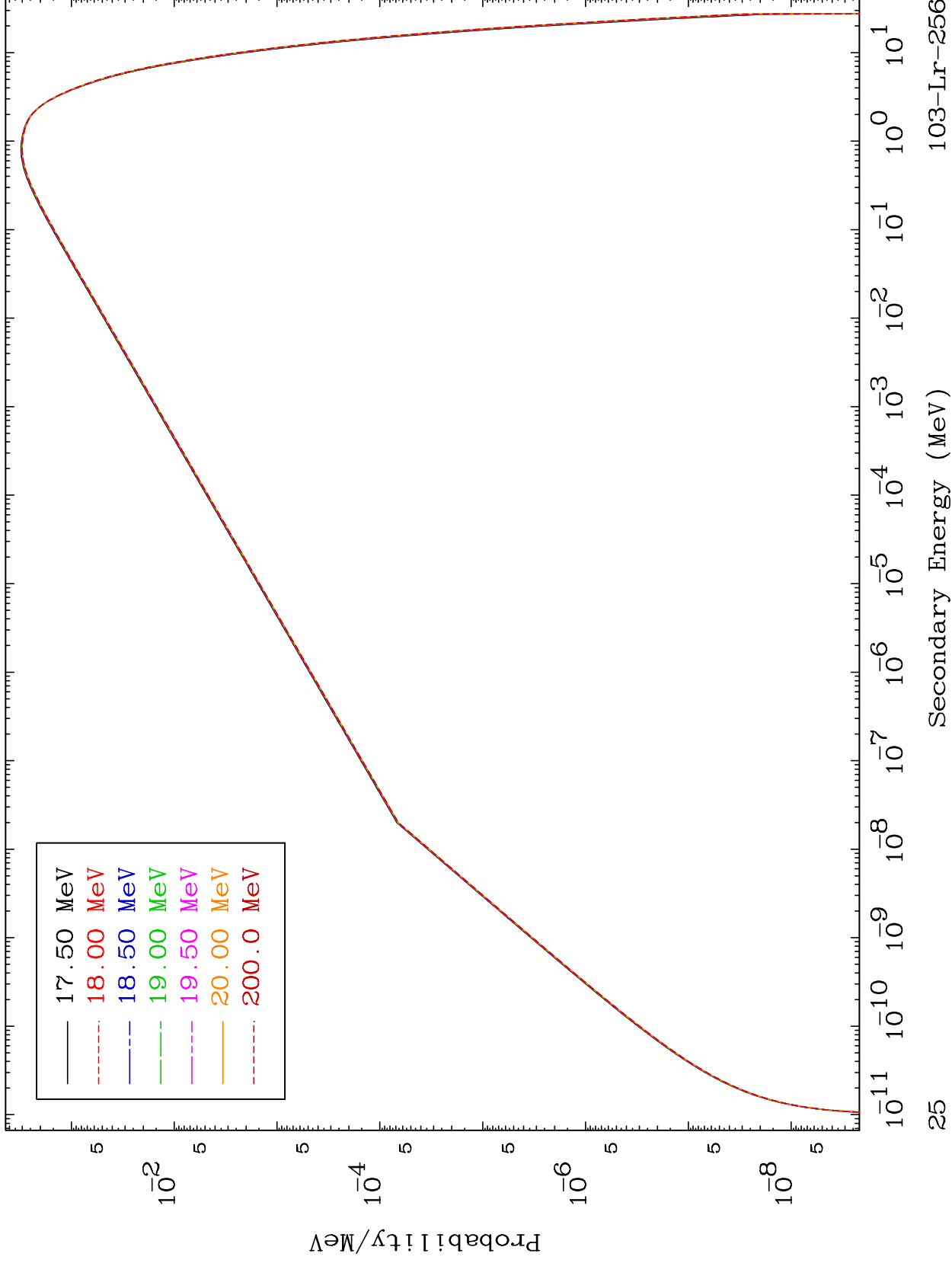
Secondary Energy (MeV)

24

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Fission
Energy Distributions

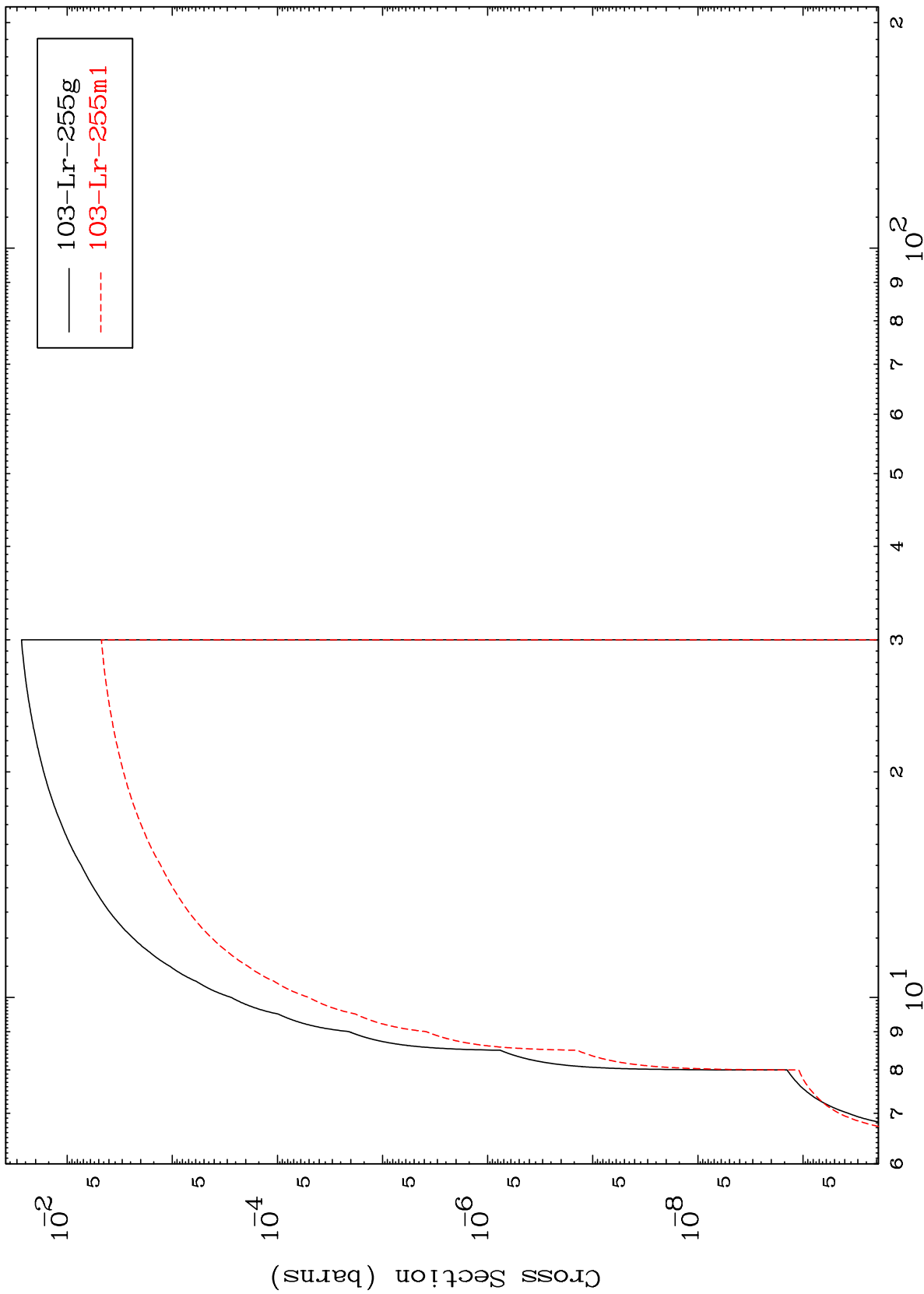
103-Lr-256



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103-Lr-256

(n,2n)
Radionuclide Production Cross Section



26

Incident Energy (MeV)

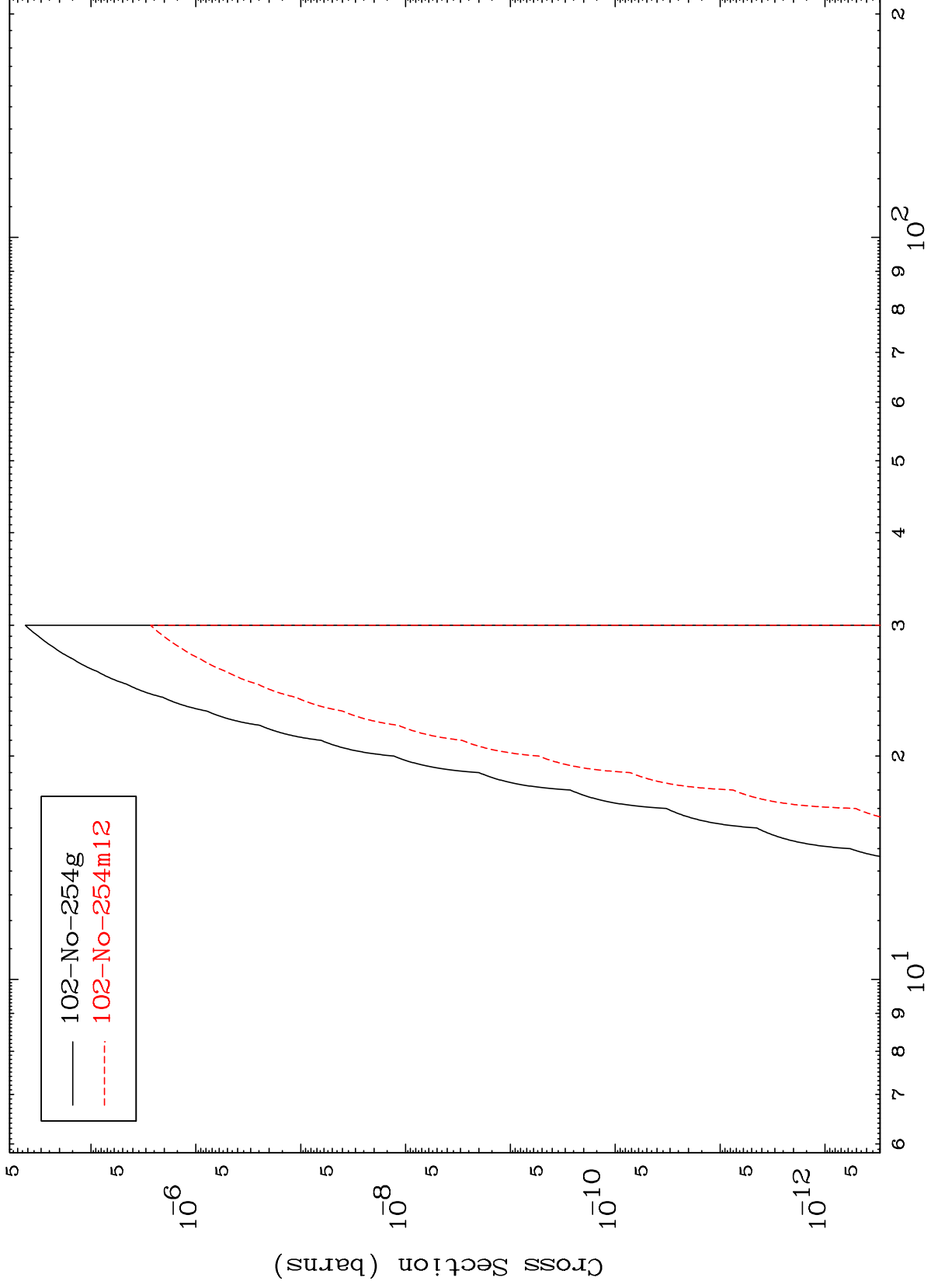
103-Lr-256

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(n,n') d

103-Lr-256

Radionuclide Production Cross Section



27

Incident Energy (MeV)

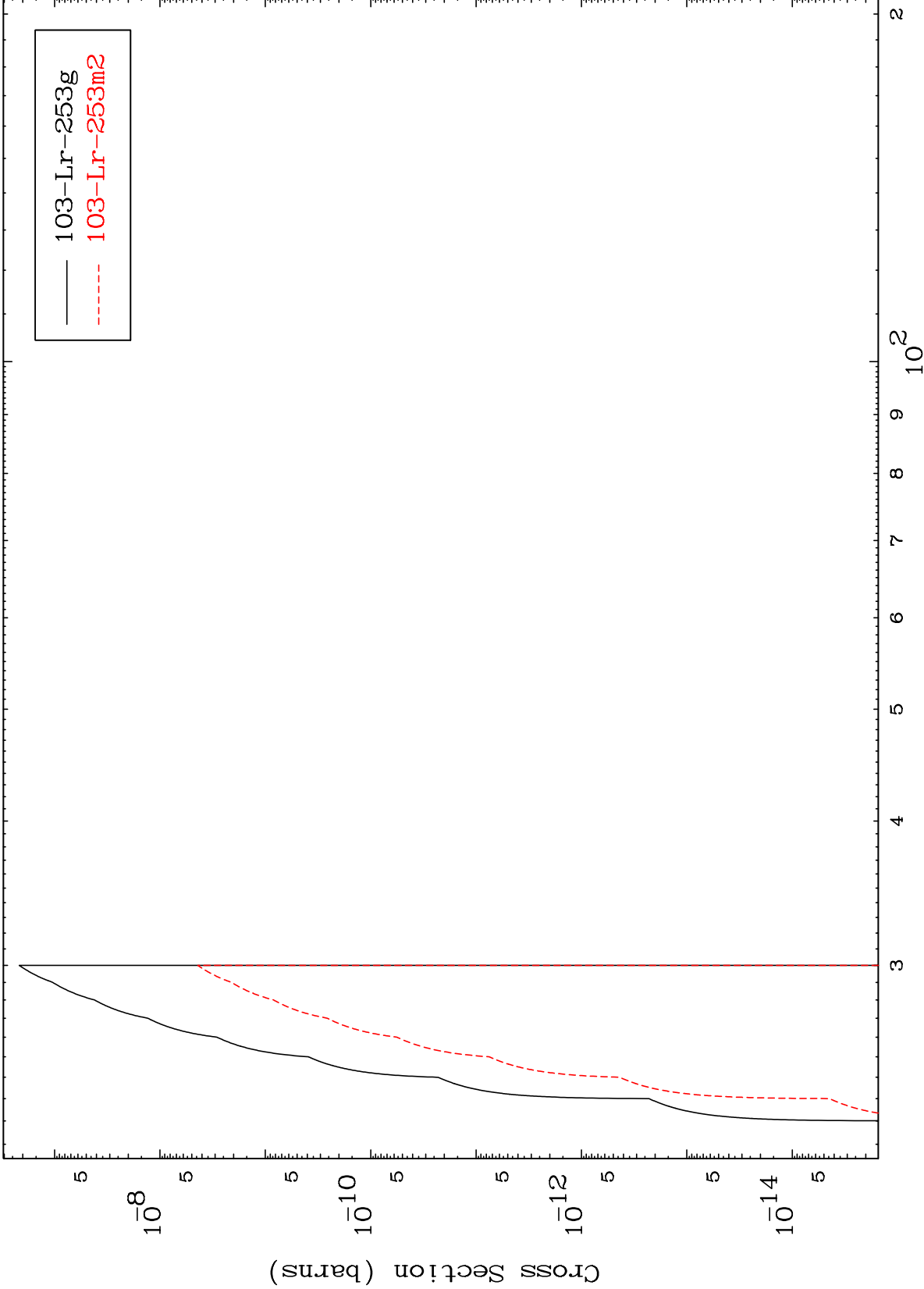
103-Lr-256

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(n,4n)

103-Lr-256

Radionuclide Production Cross Section



28

Incident Energy (MeV)

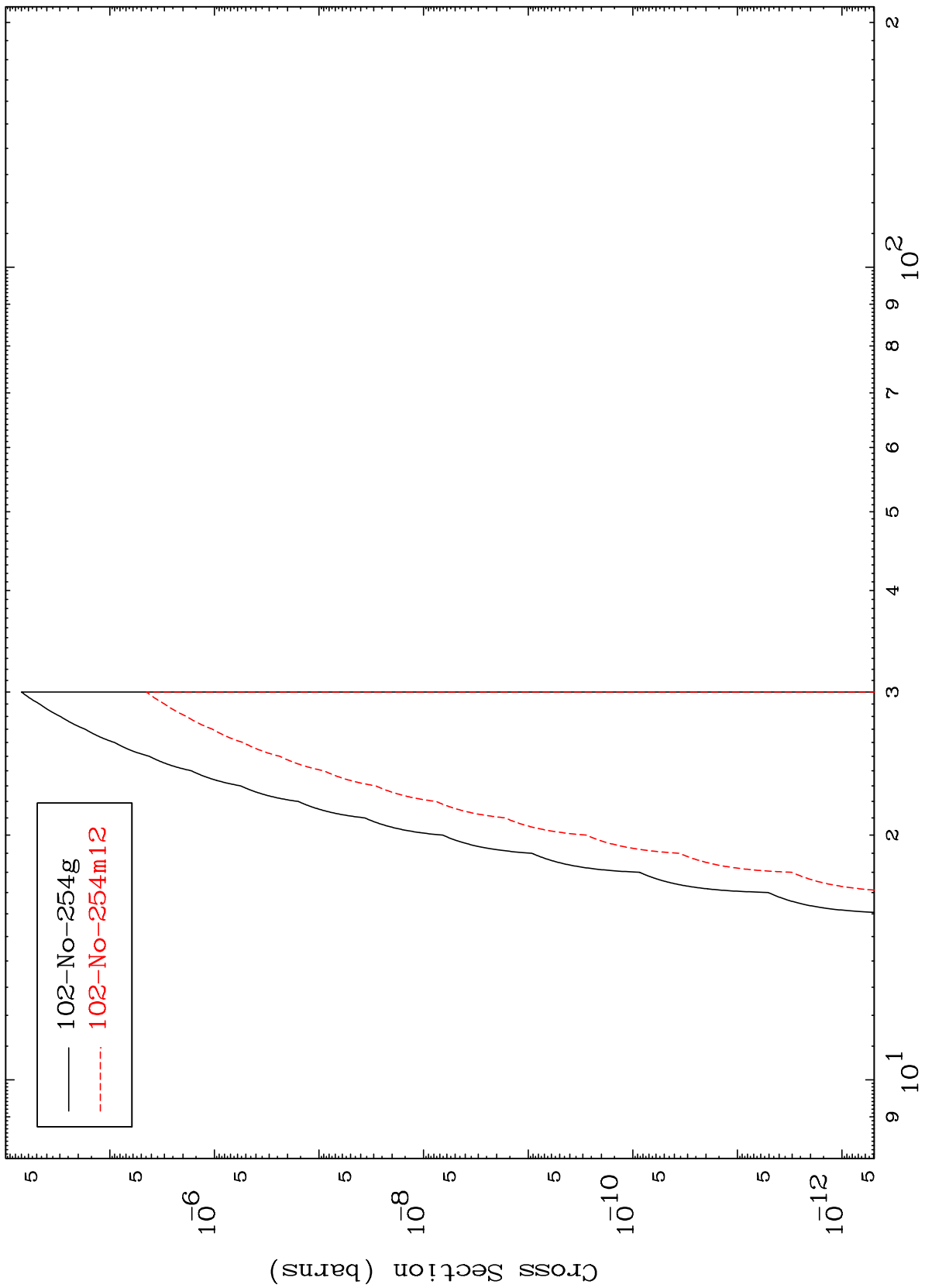
103-Lr-256

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(n,2n) p

103-Lr-256

Radionuclide Production Cross Section



29

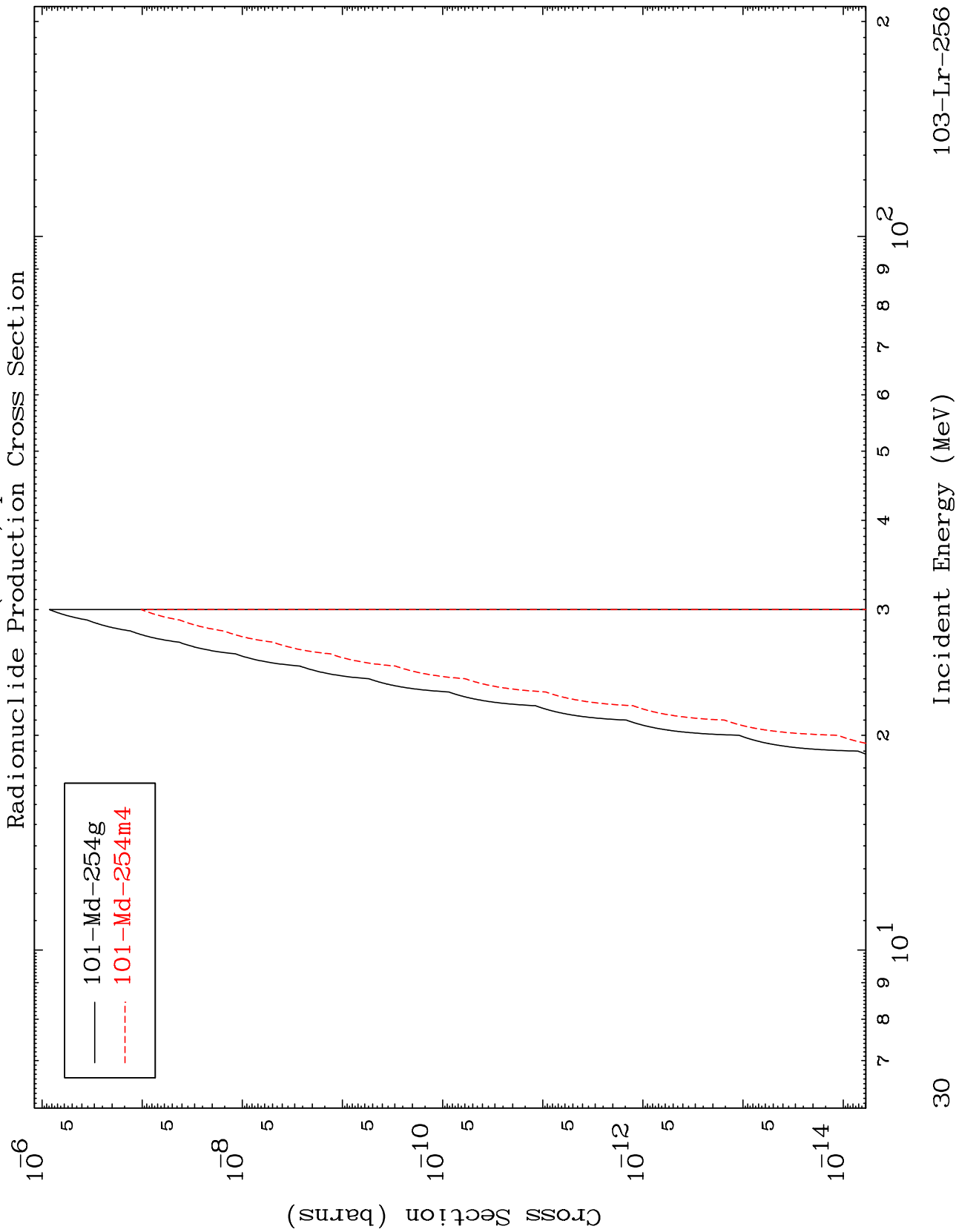
Incident Energy (MeV)

103-Lr-256

MAT 9985

(n,2n) p

103-Lr-256

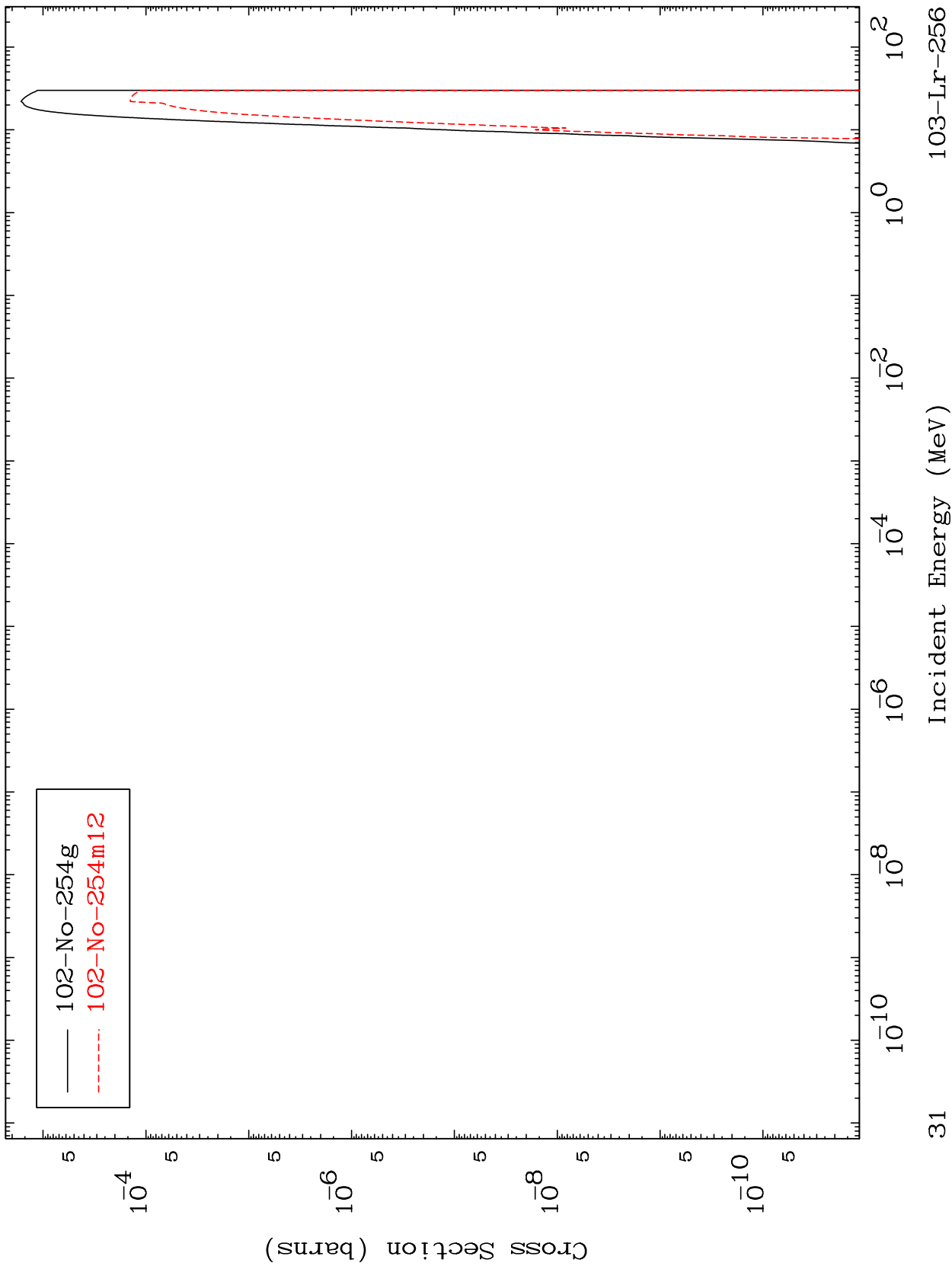


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(n, t)

103-Lr-256

Radionuclide Production Cross Section

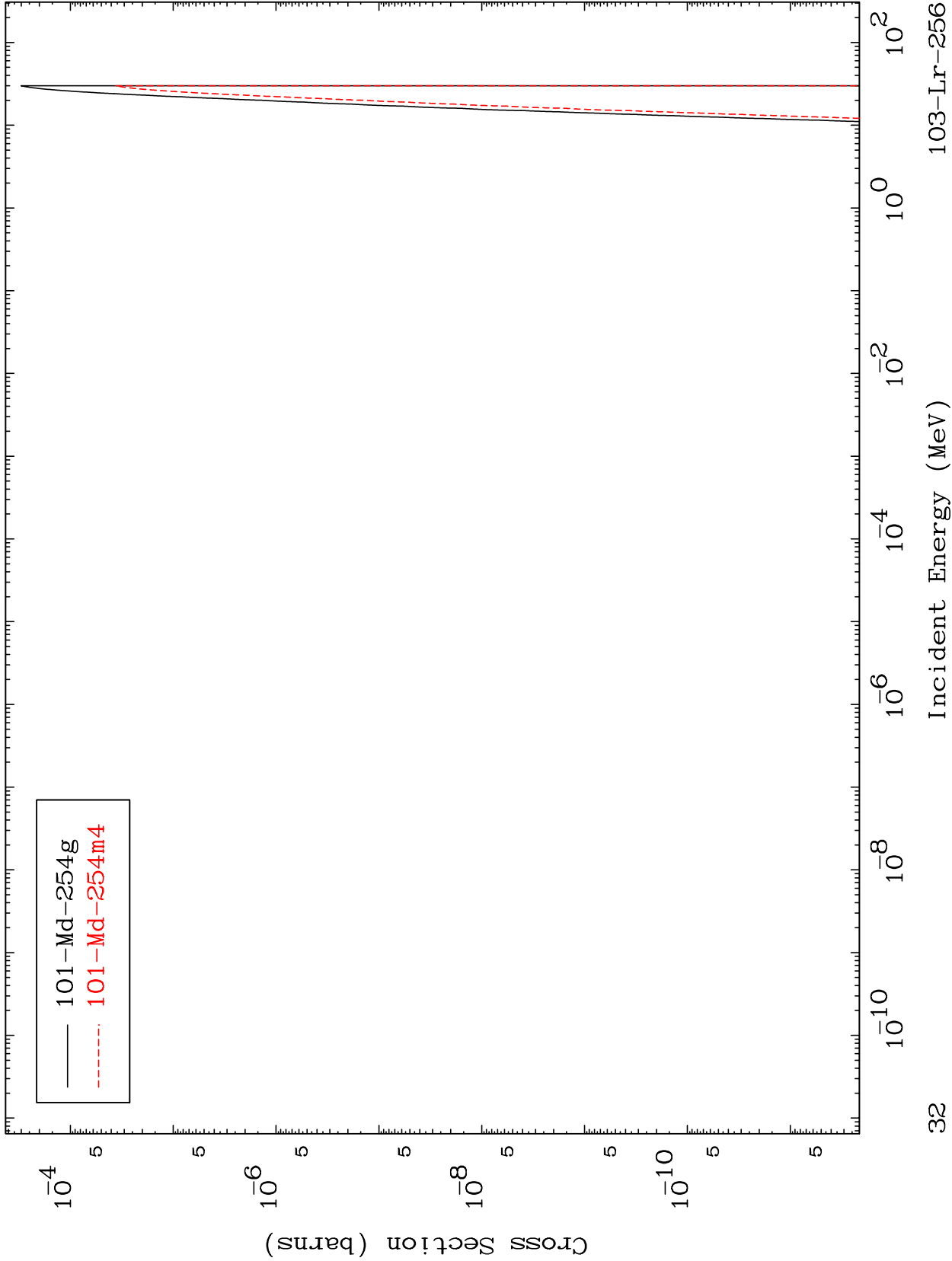


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(n,He-3)

103-Lr-256

Radionuclide Production Cross Section



MAT 9985

(n,p) d

103-Lr-256

Radionuclide Production Cross Section

