

Program EVALPLOT
(Version 2021-1)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

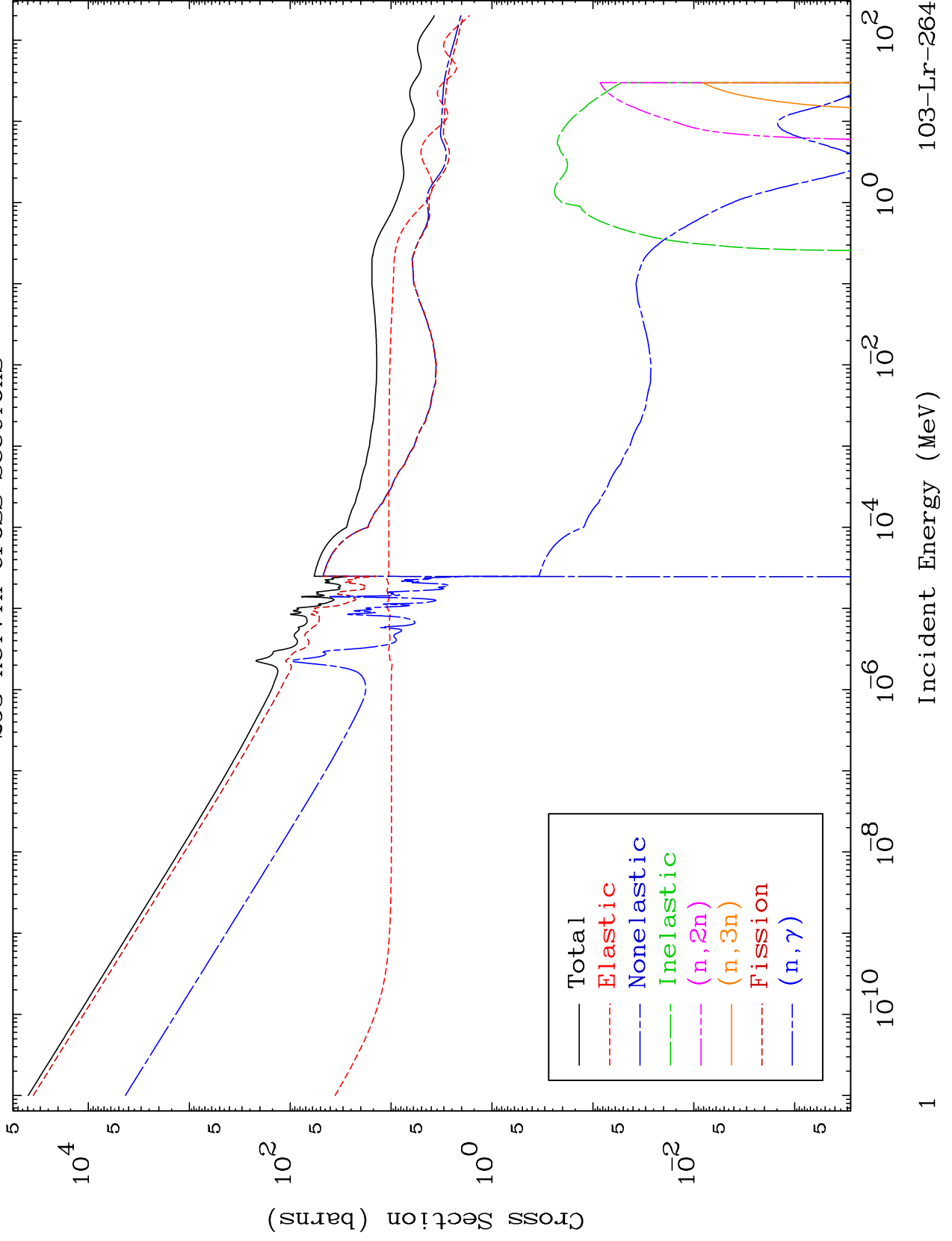
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9993

Neutron Major
293 Kelvin Cross Sections

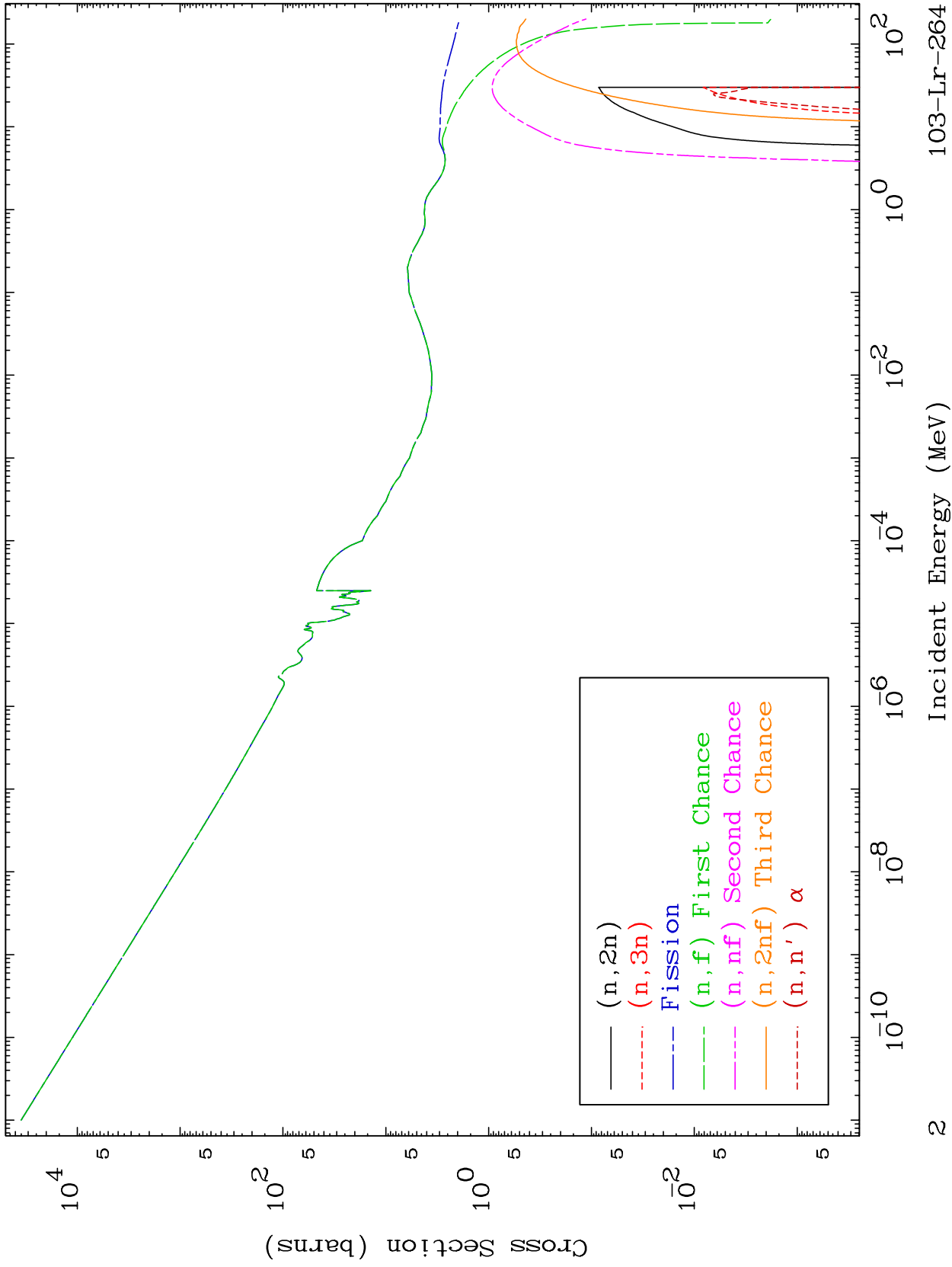
103-Lr-264



MAT 9993

Neutron Absorption
293 Kelvin Cross Sections

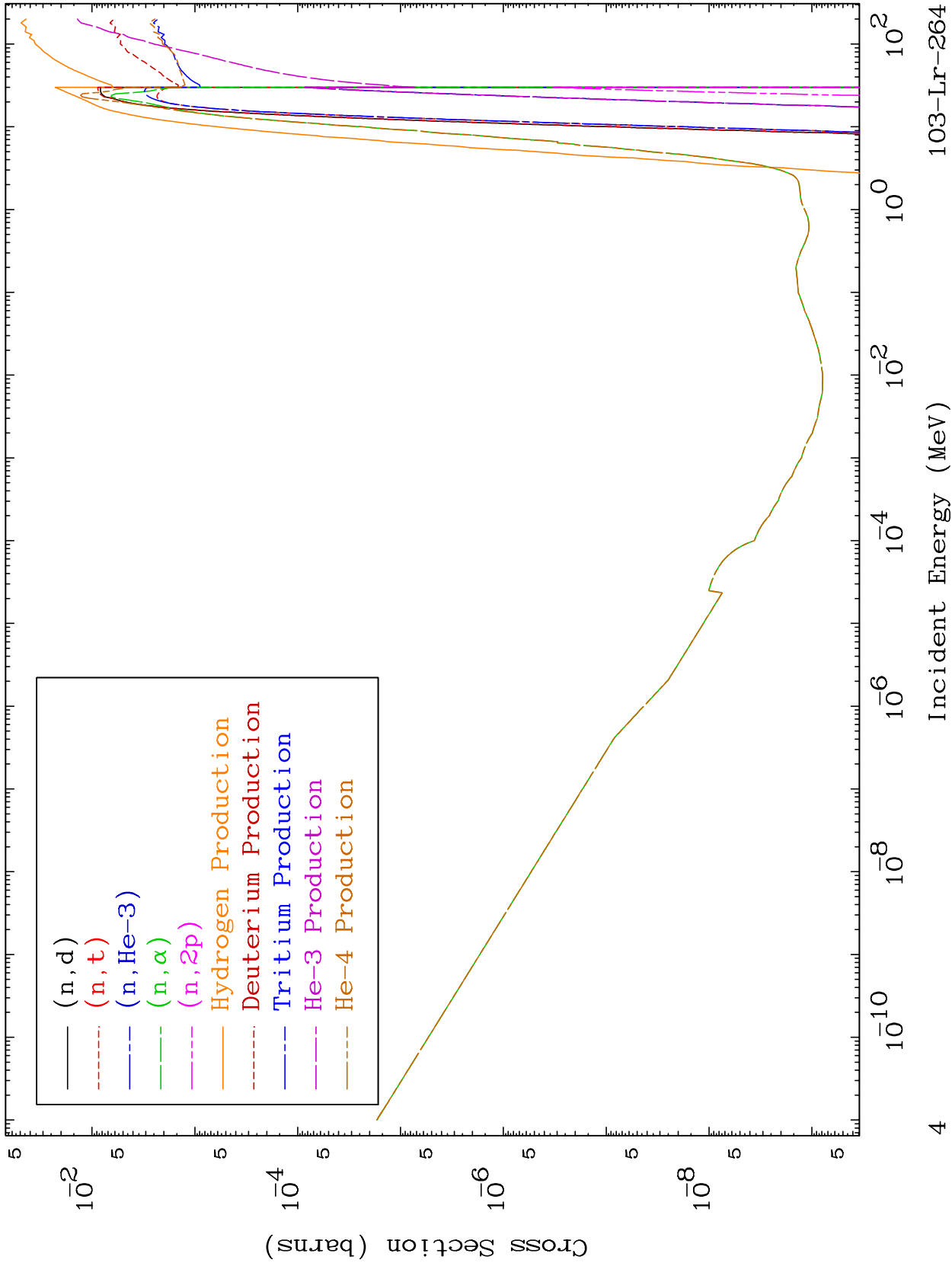
103-Lr-264



MAT 9993

Neutron Absorption
293 Kelvin Cross Sections

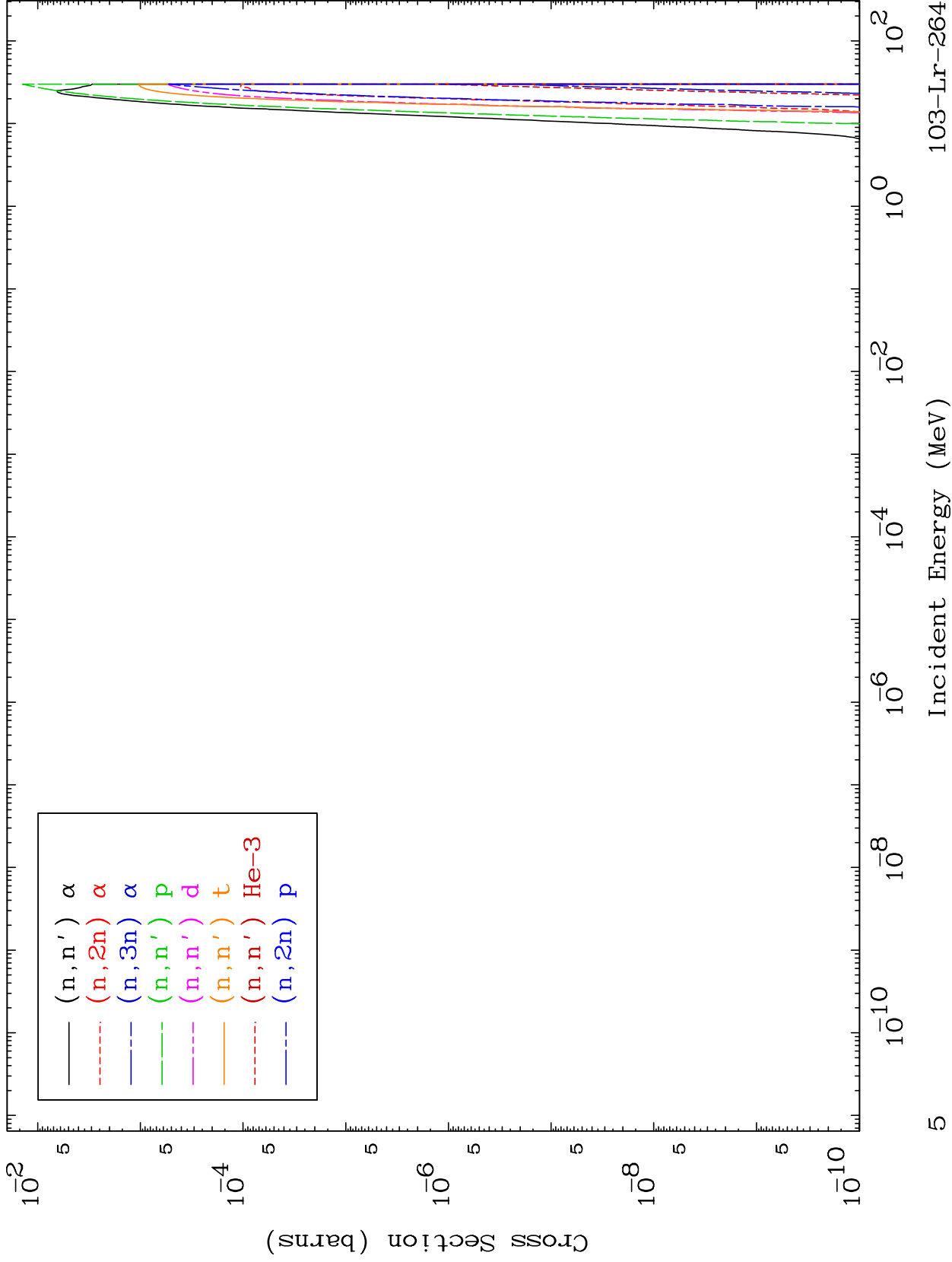
103-Lr-264



MAT 9993

Charged Particle
293 Kelvin Cross Sections

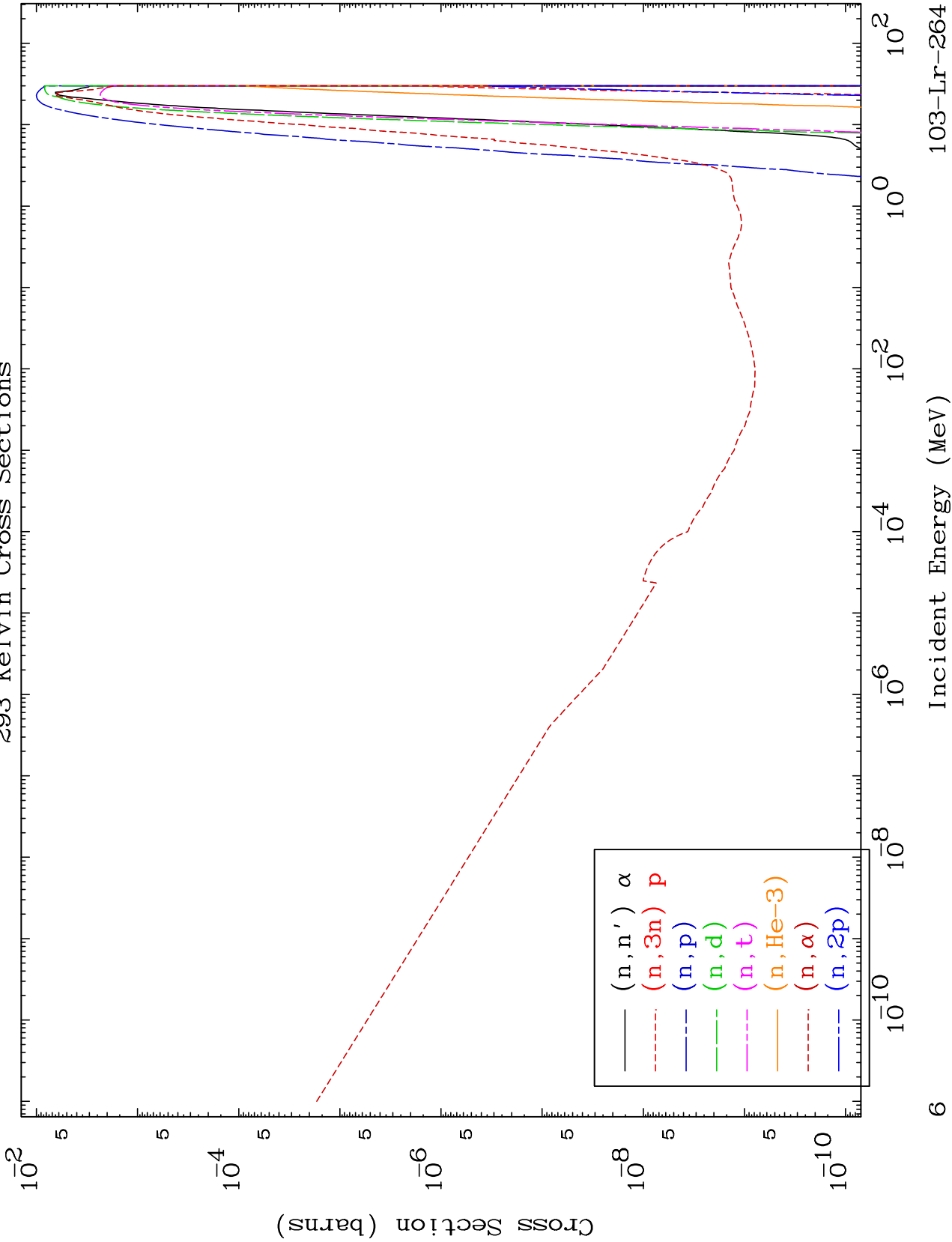
103-Lr-264



MAT 9993

Charged Particle
293 Kelvin Cross Sections

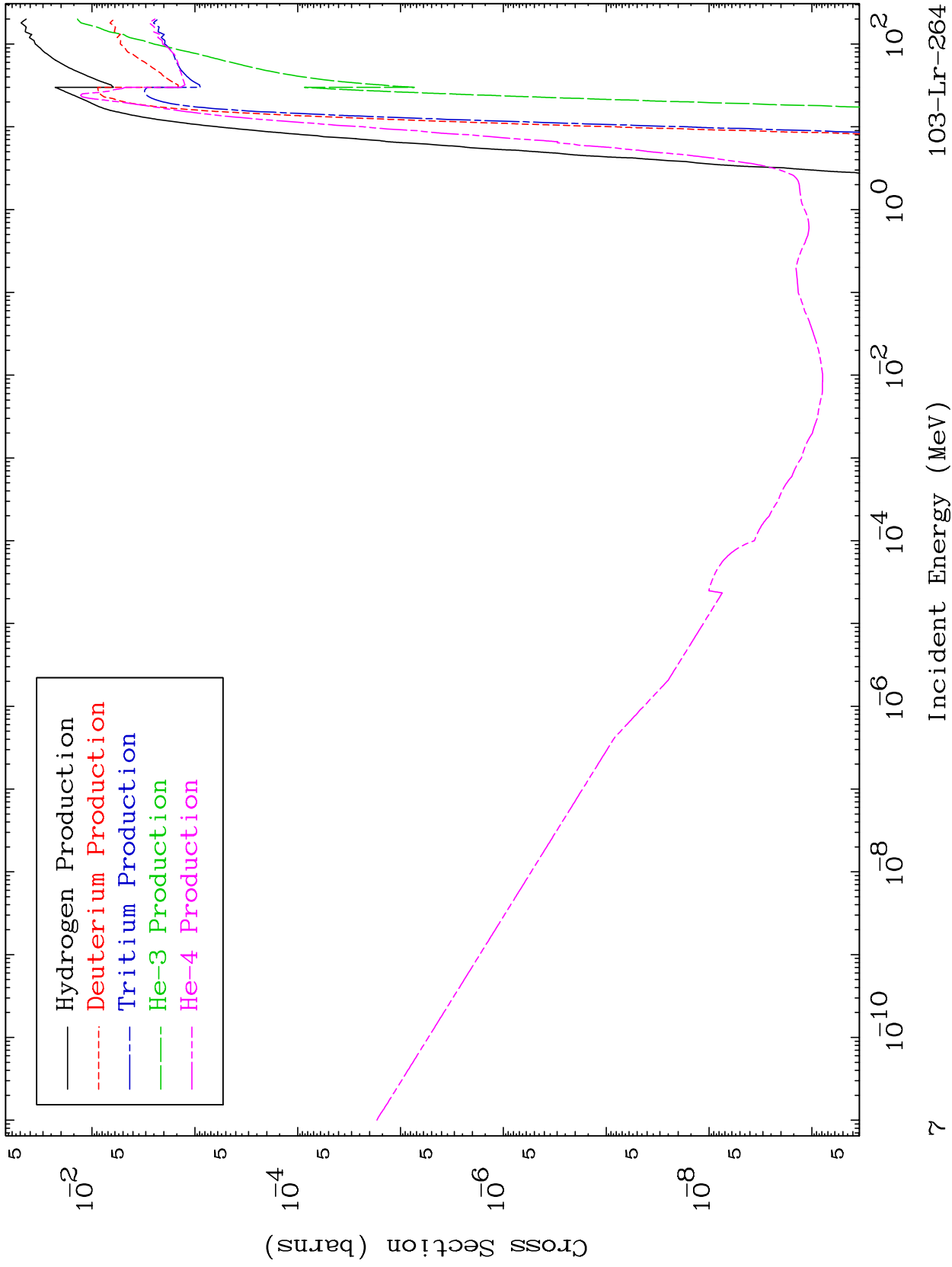
103-Lr-264



MAT 9993

Particle Production
293 Kelvin Cross Sections

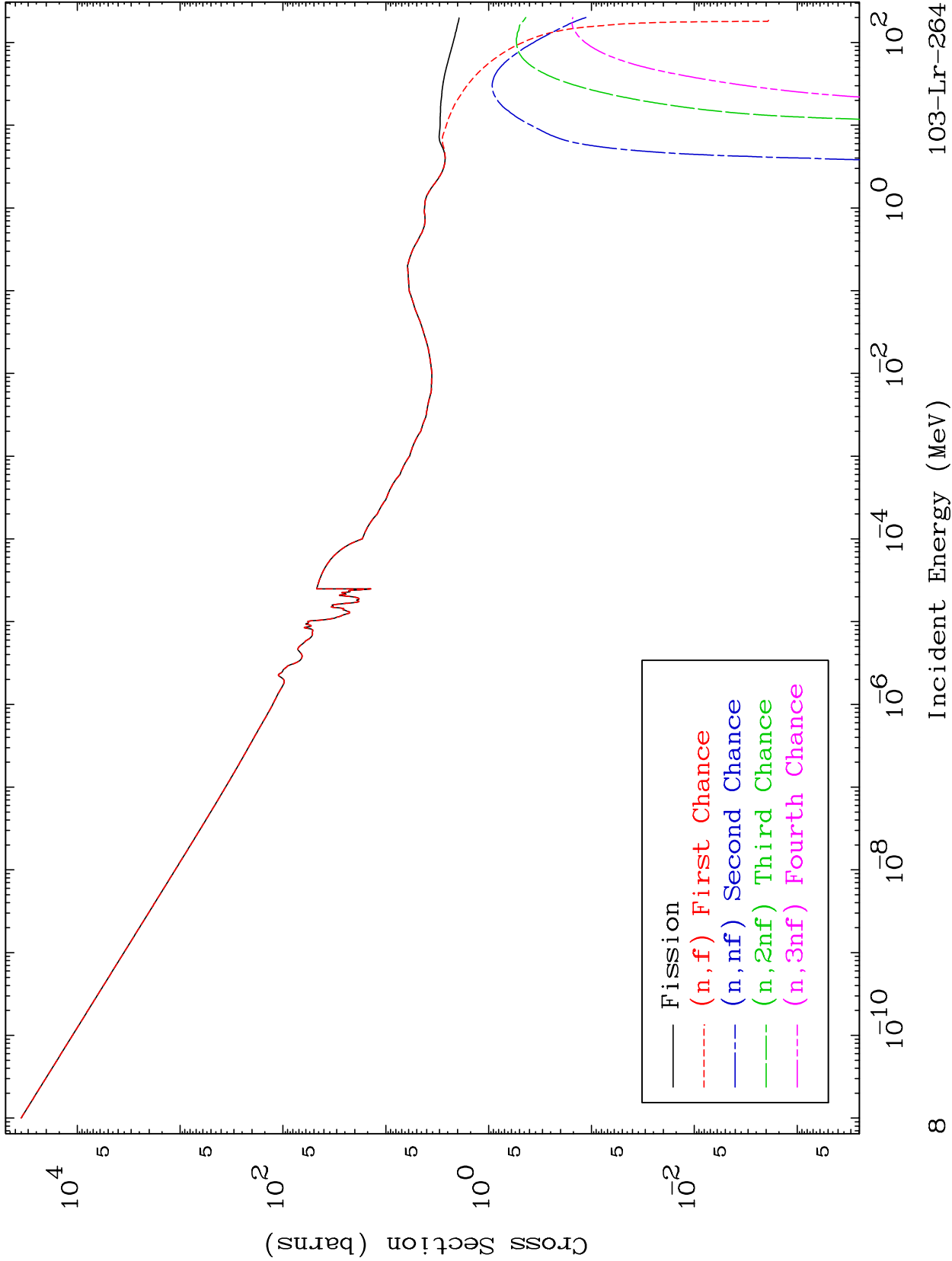
103-Lr-264



MAT 9993

Fission
293 Kelvin Cross Sections

103-Lr-264

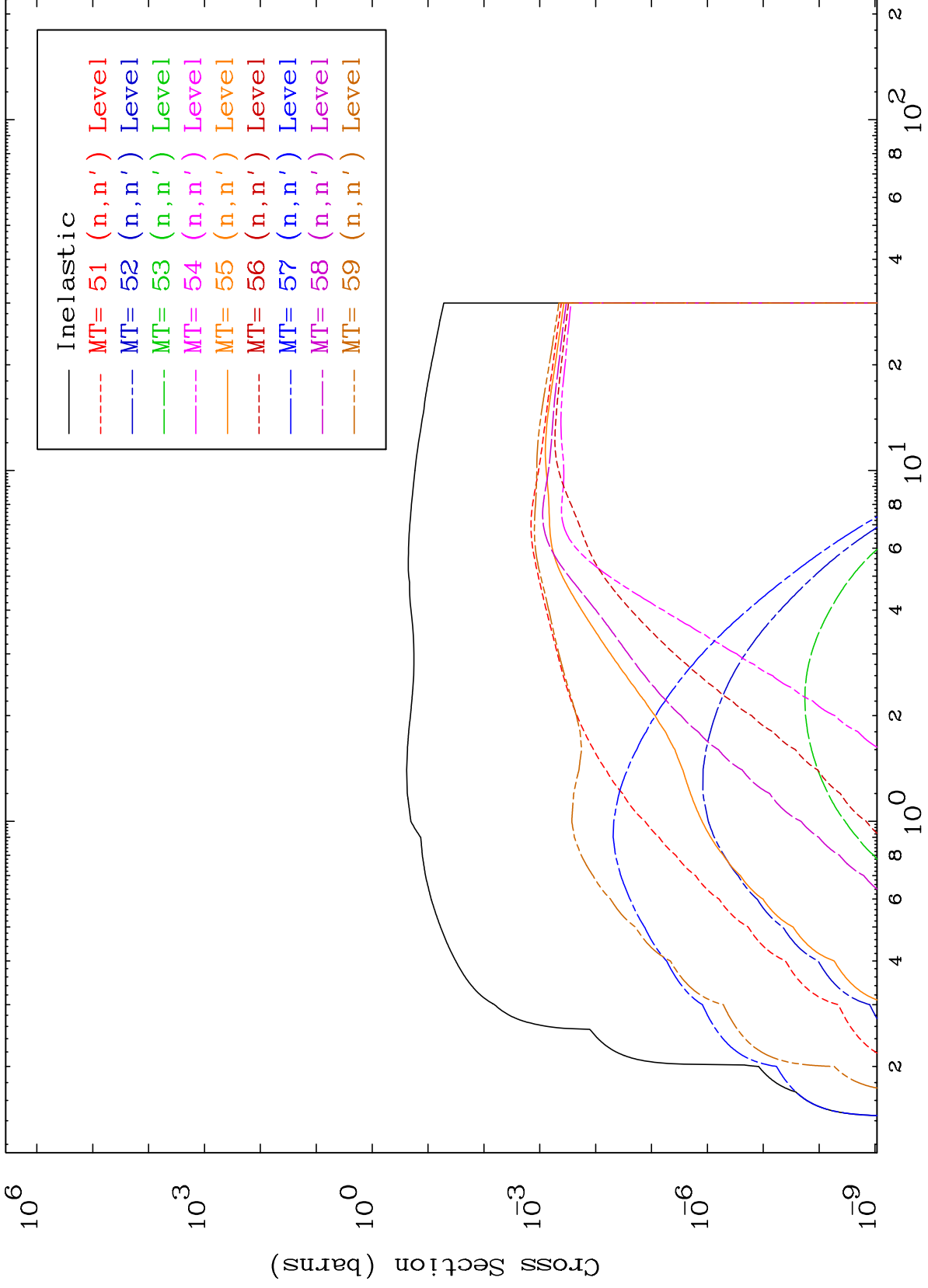


MAT 9993

(n,n') Levels

103-Lr-264

293 Kelvin Cross Sections

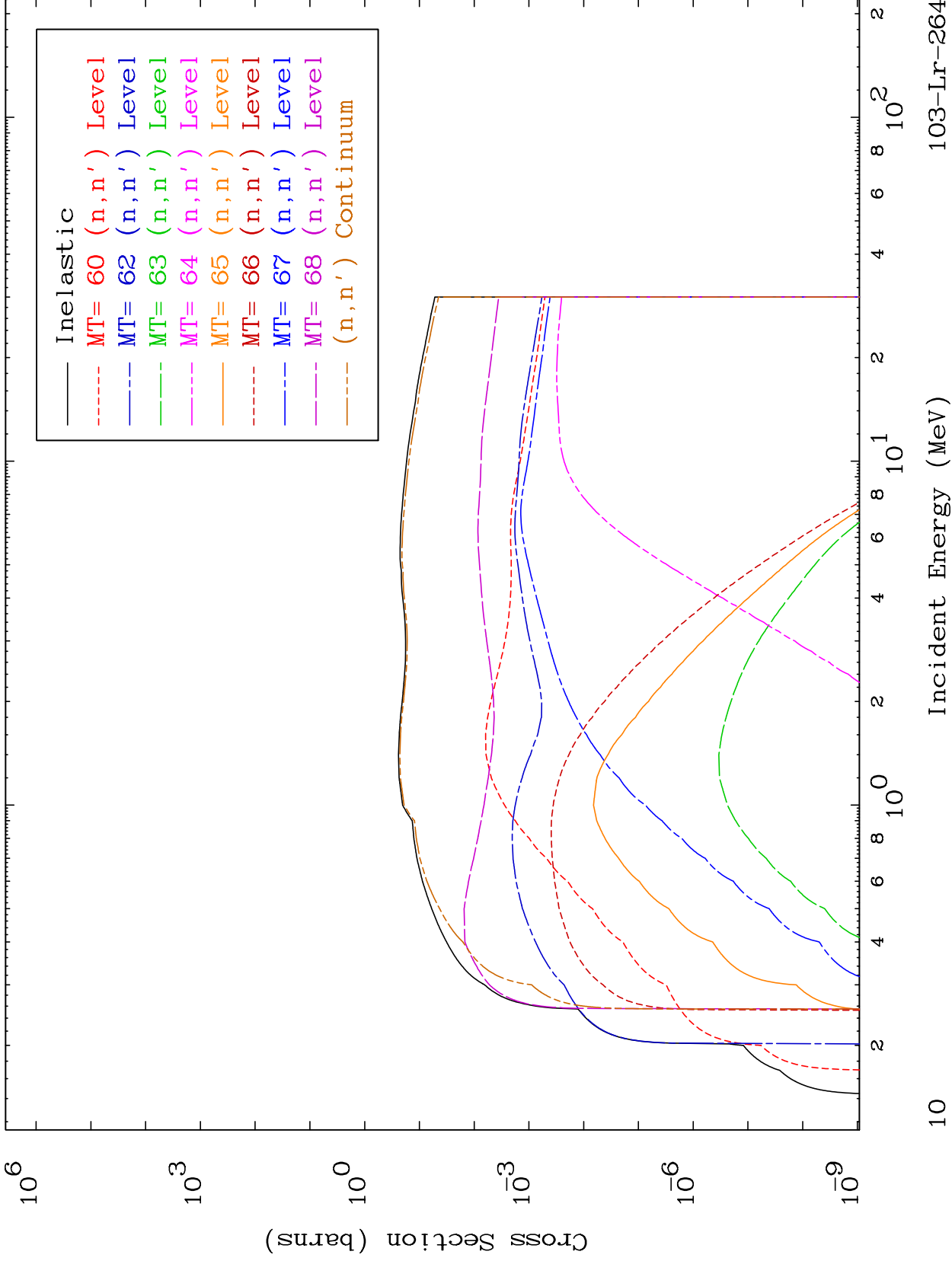


MAT 9993

(n,n') Levels

103-Lr-264

293 Kelvin Cross Sections



10

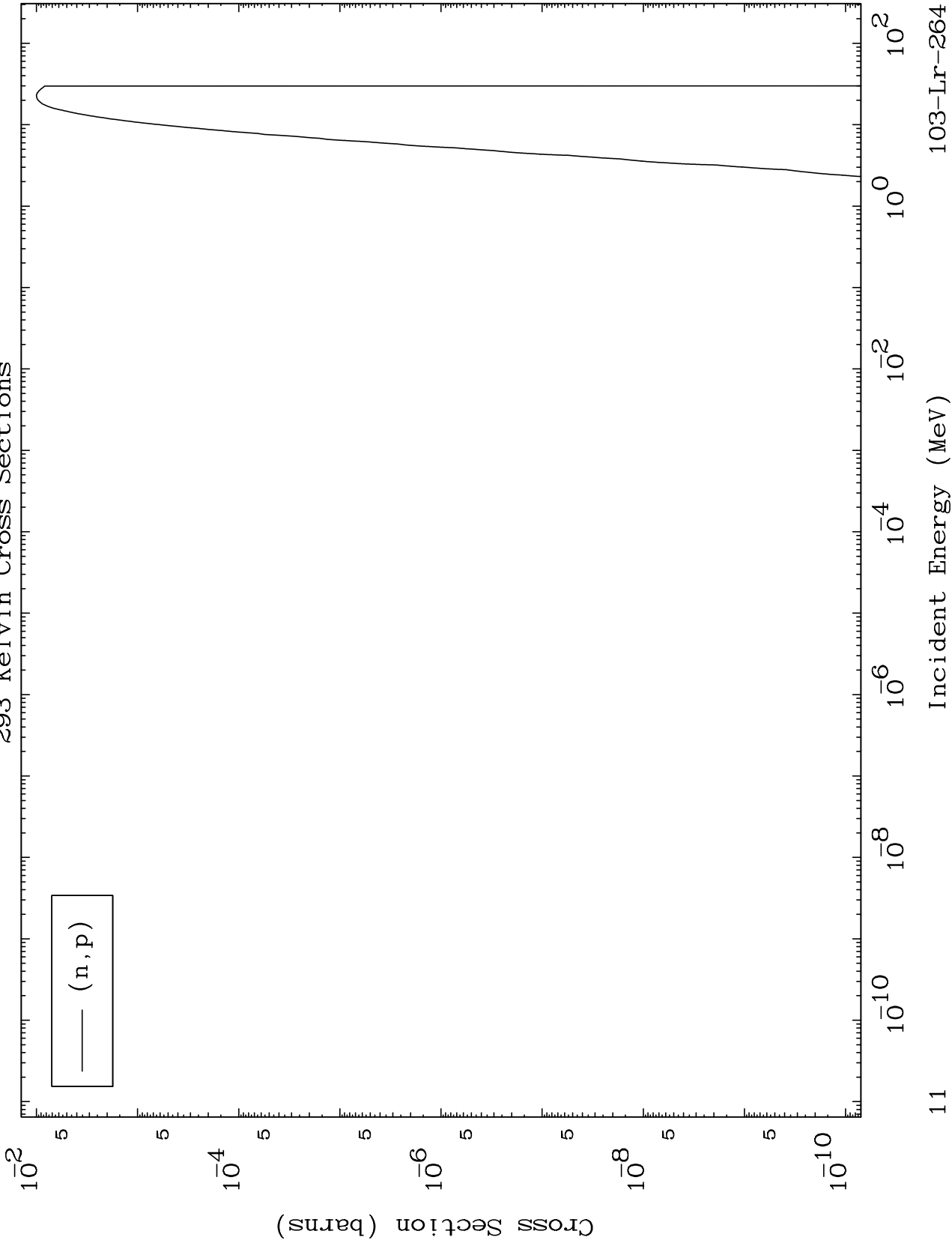
Incident Energy (MeV)

103-Lr-264

MAT 9993

(n,p) Levels
293 Kelvin Cross Sections

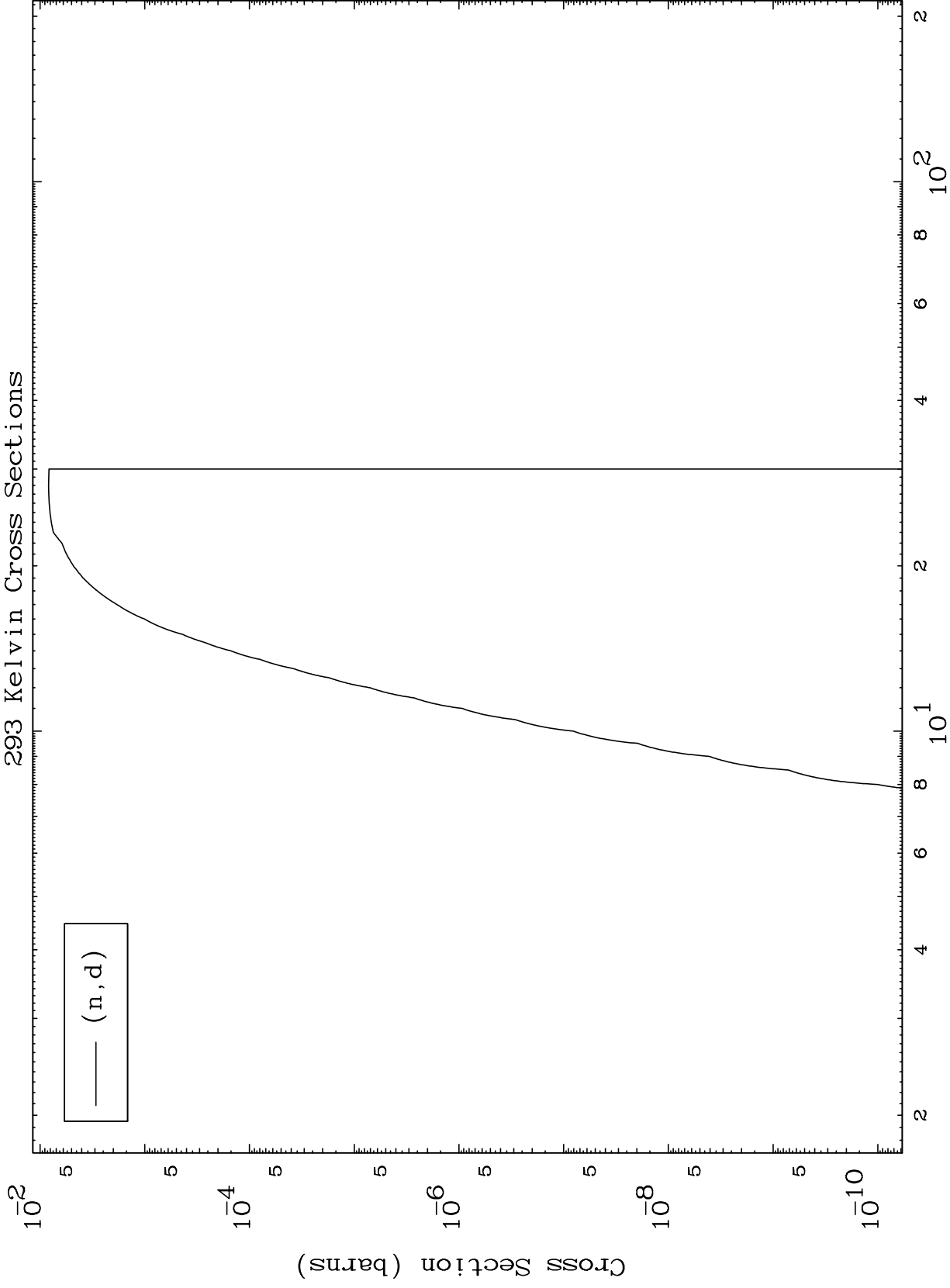
103-Lr-264



MAT 9993

(n,d) Levels
293 Kelvin Cross Sections

103-Lr-264



12

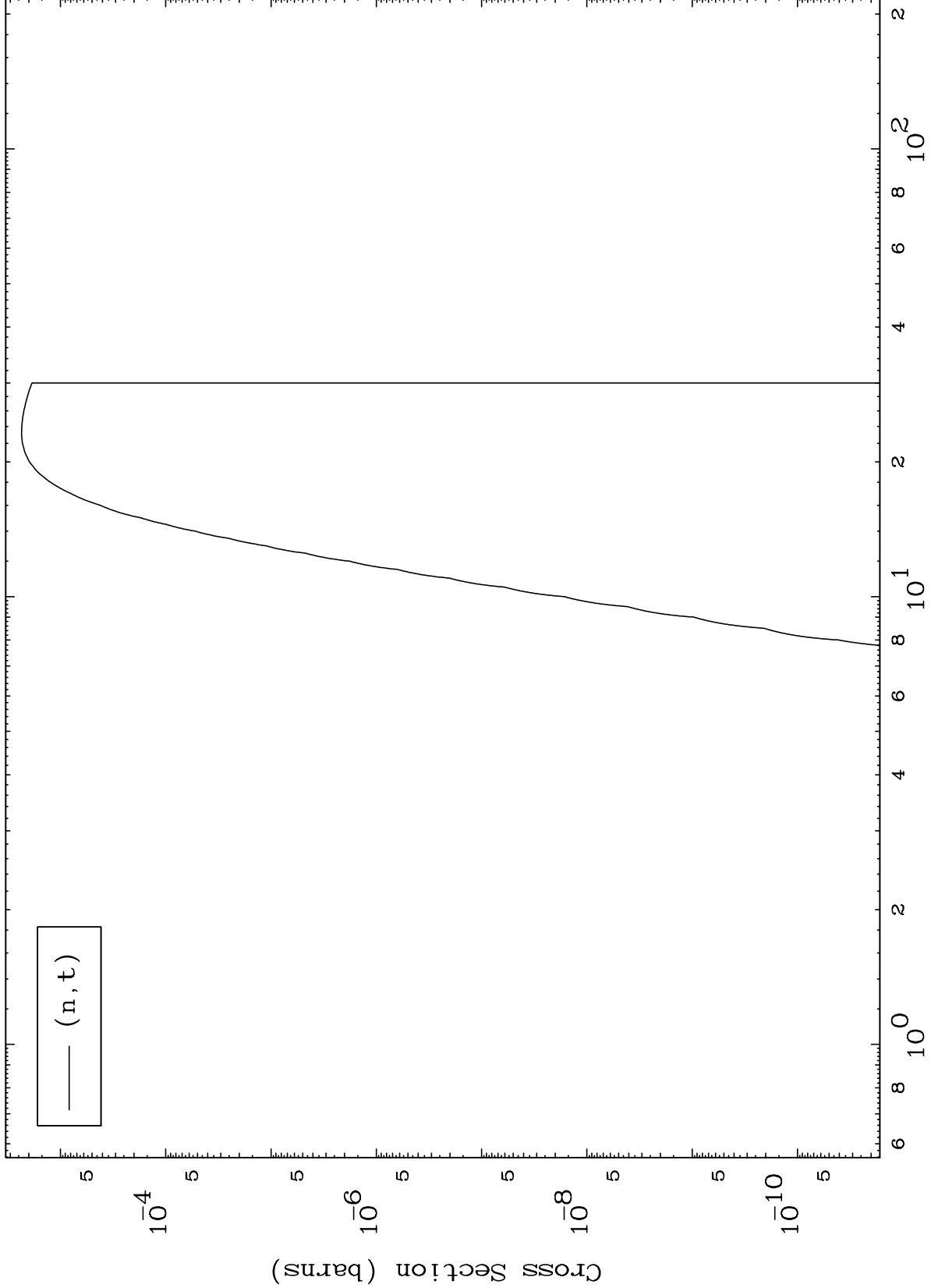
Incident Energy (MeV)

103-Lr-264

MAT 9993

(n,t) Levels
293 Kelvin Cross Sections

103-Lr-264



13

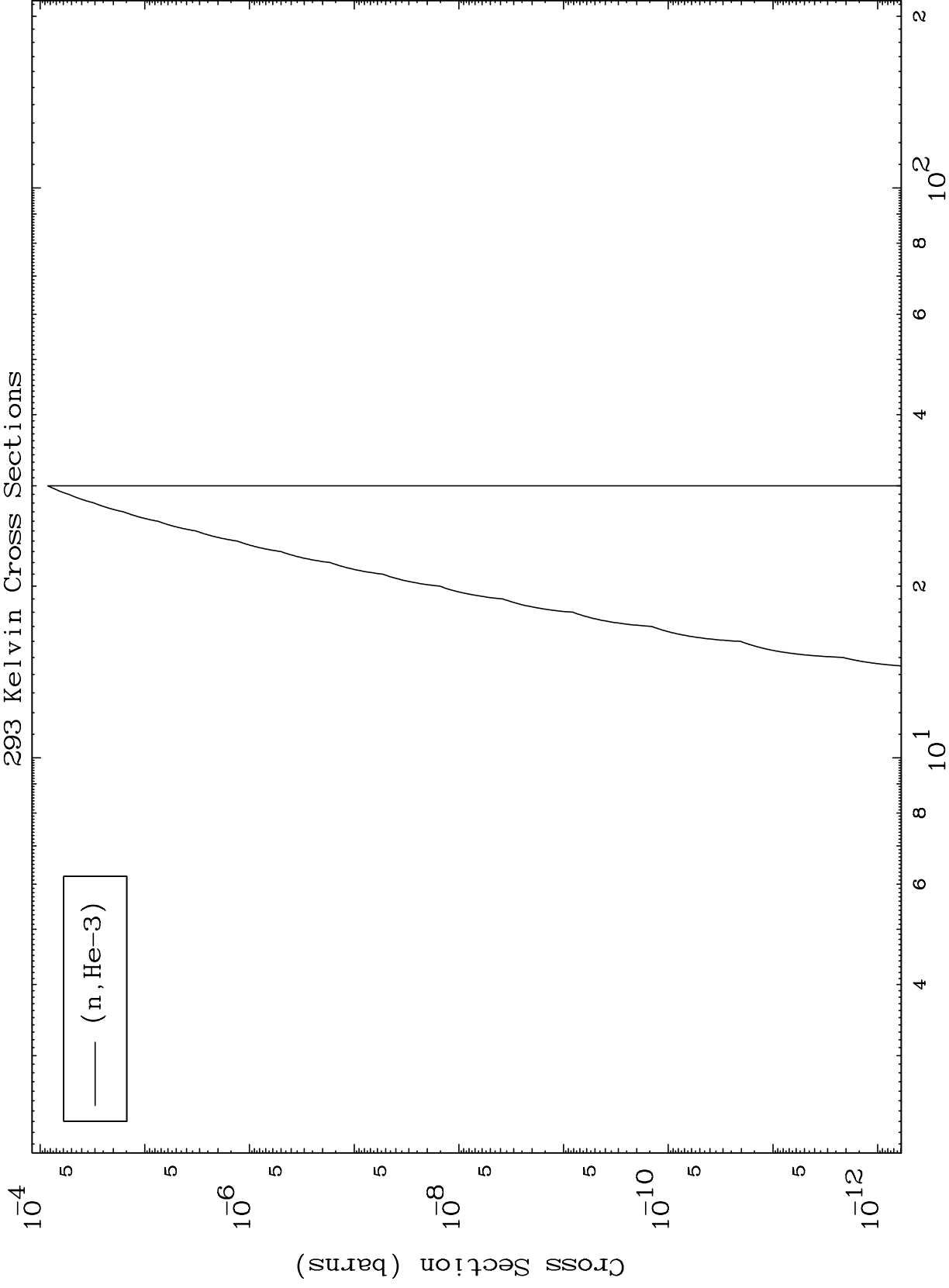
Incident Energy (MeV)

103-Lr-264

MAT 9993

(n,He3) Levels
293 Kelvin Cross Sections

103-Lr-264



14

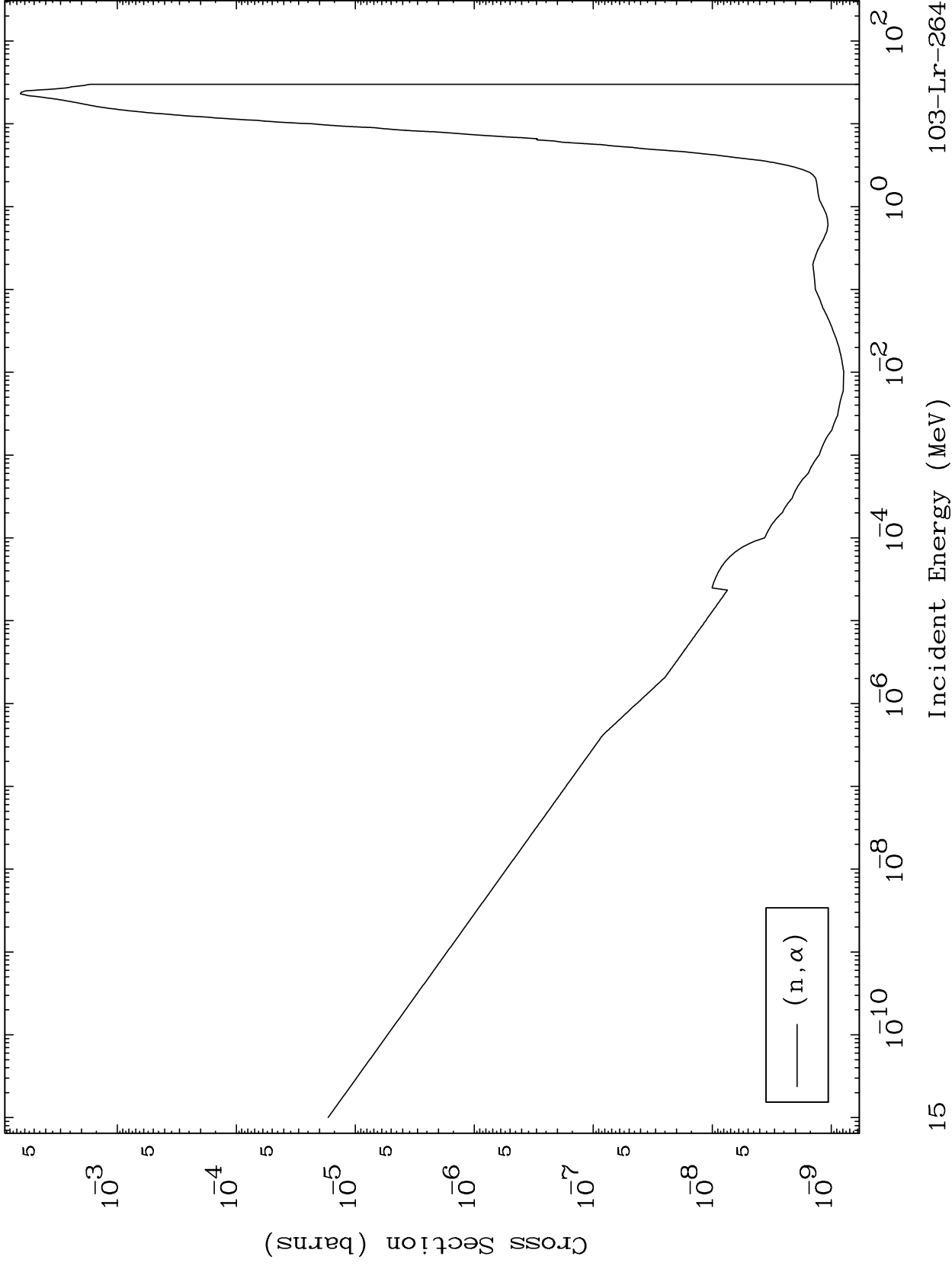
Incident Energy (MeV)

103-Lr-264

MAT 9993

(n, α) Levels
293 Kelvin Cross Sections

103-Lr-264

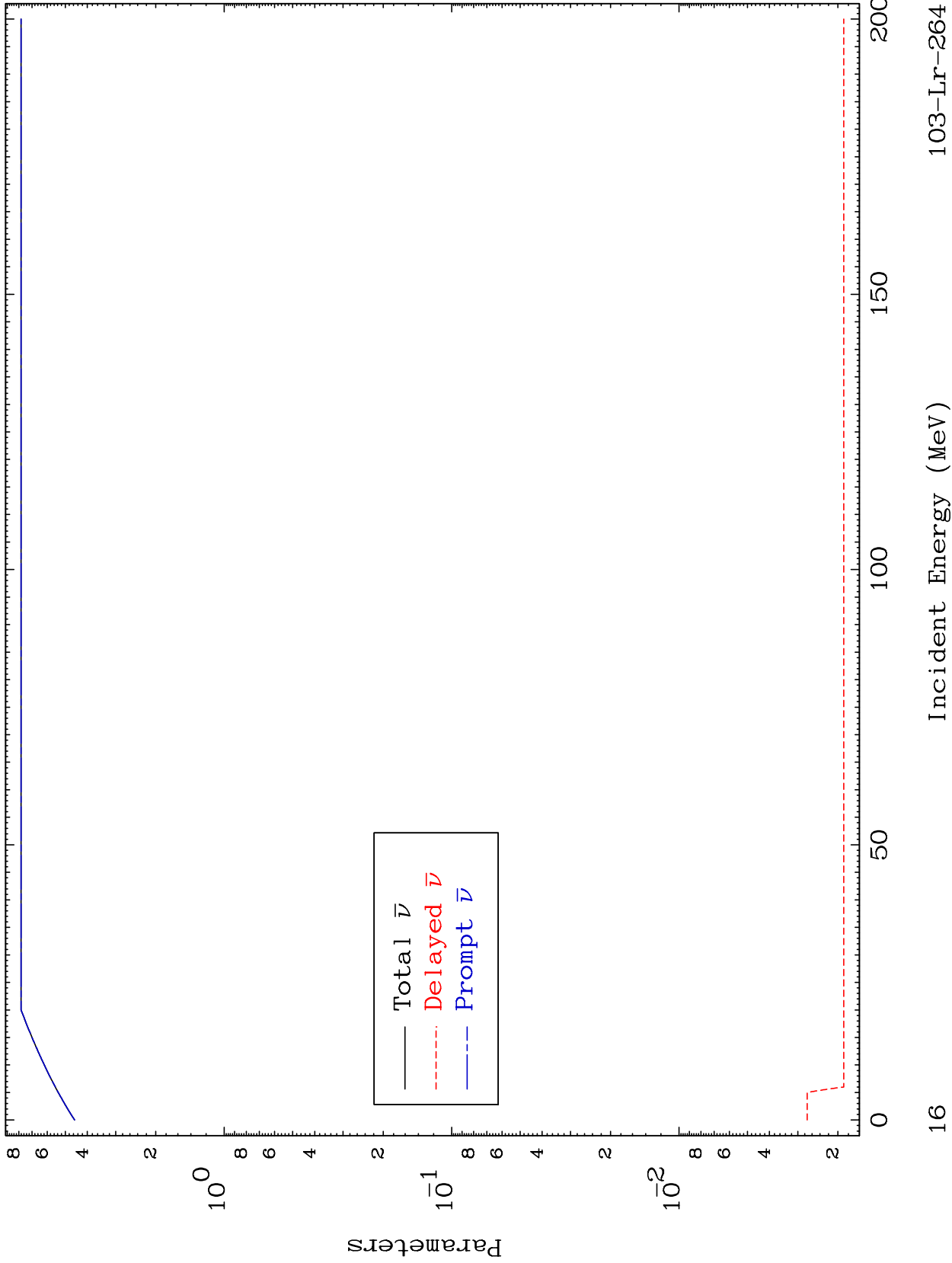


15

MAT 9993

Energy Release
Parameters

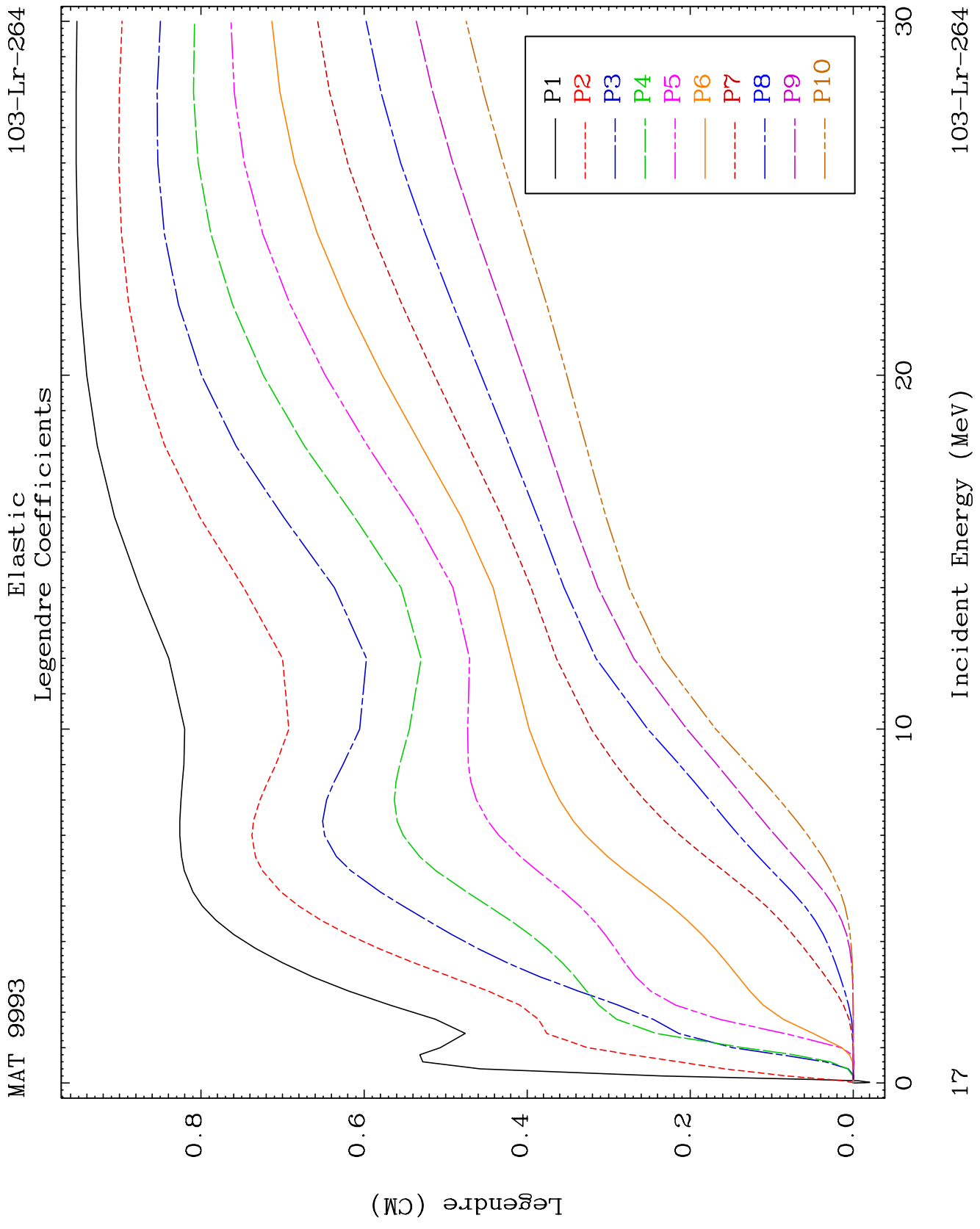
103-Lr-264

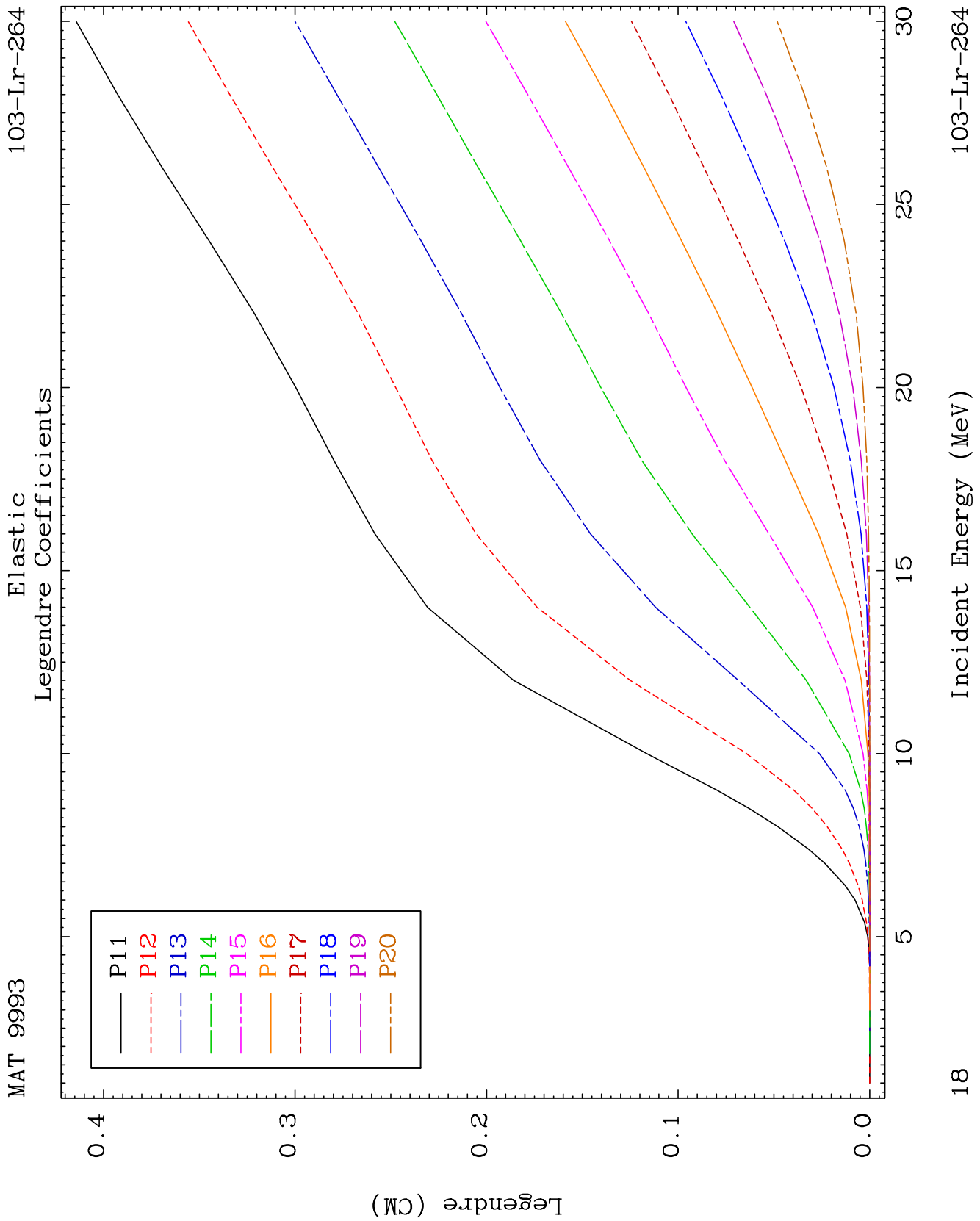


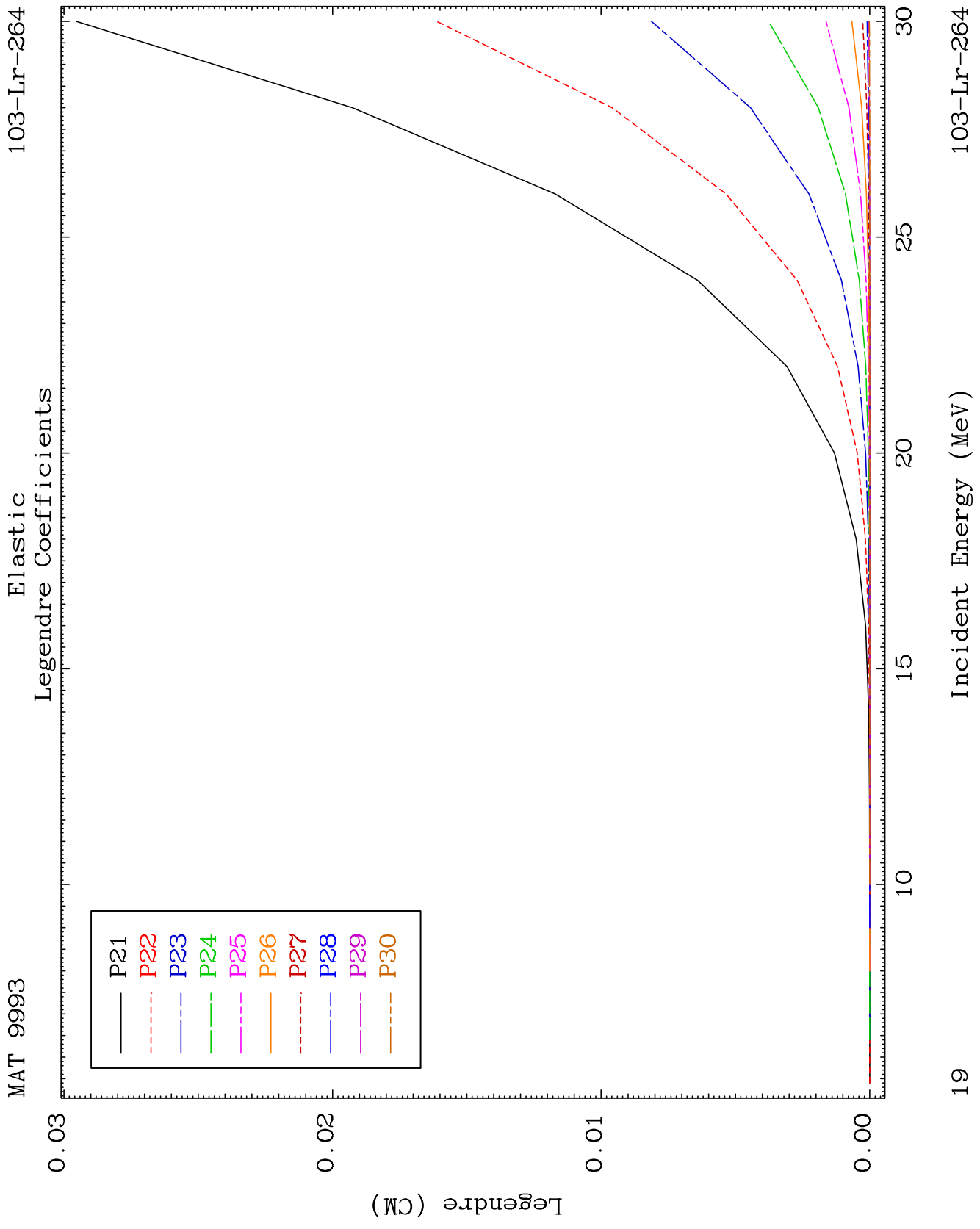
16

Incident Energy (MeV)

103-Lr-264



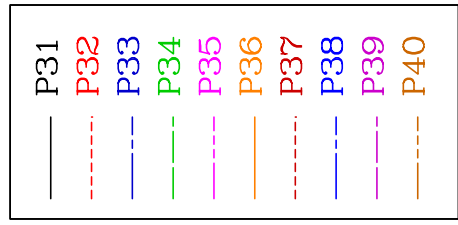




MAT 9993

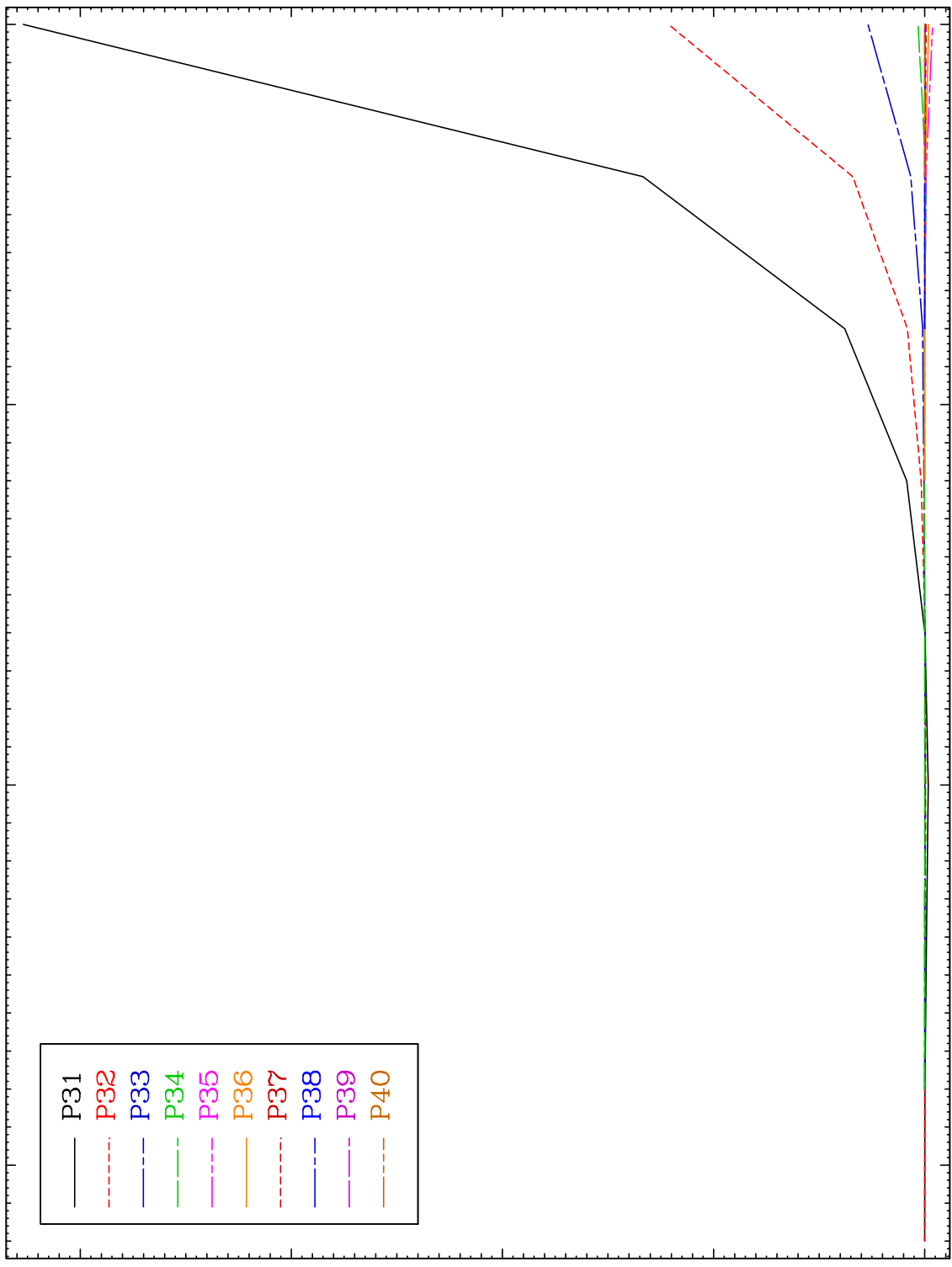
Elastic
Legendre Coefficients

103-Lr-264



$\times 10^{-6}$

Legendre (CM)



30

25

20

20

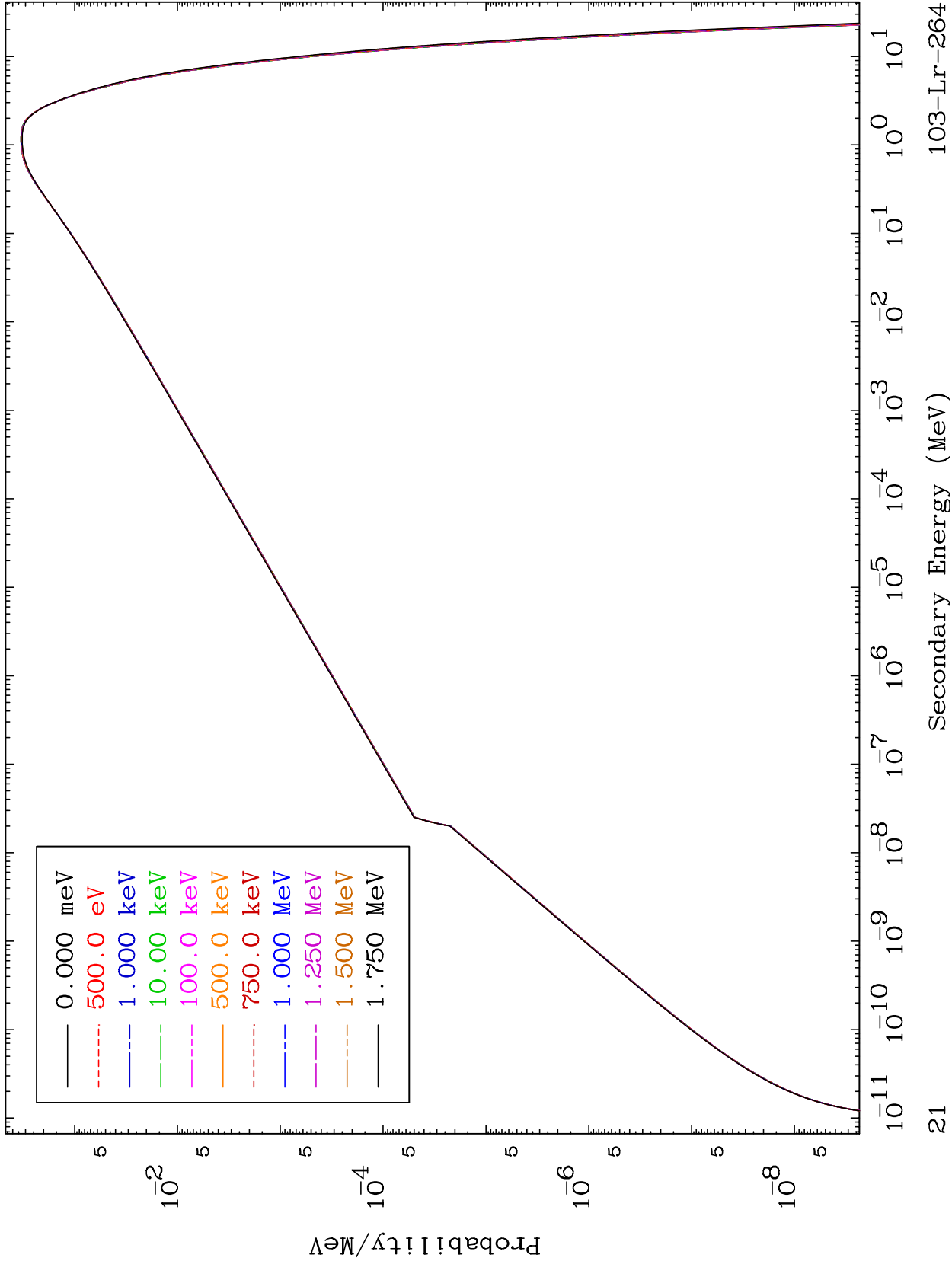
Incident Energy (MeV)

103-Lr-264

MAT 9993

Fission Energy Distributions

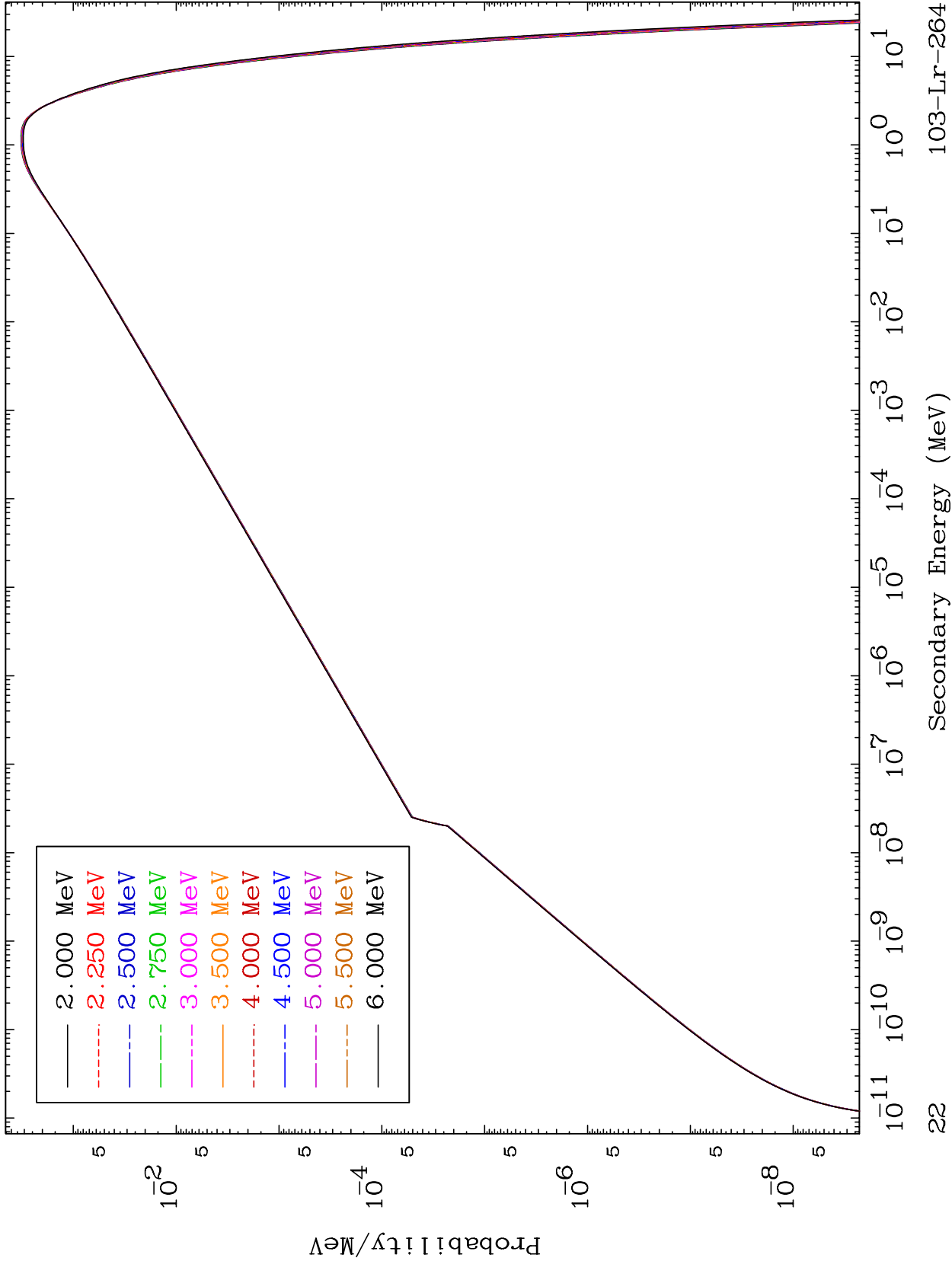
103-Lr-264



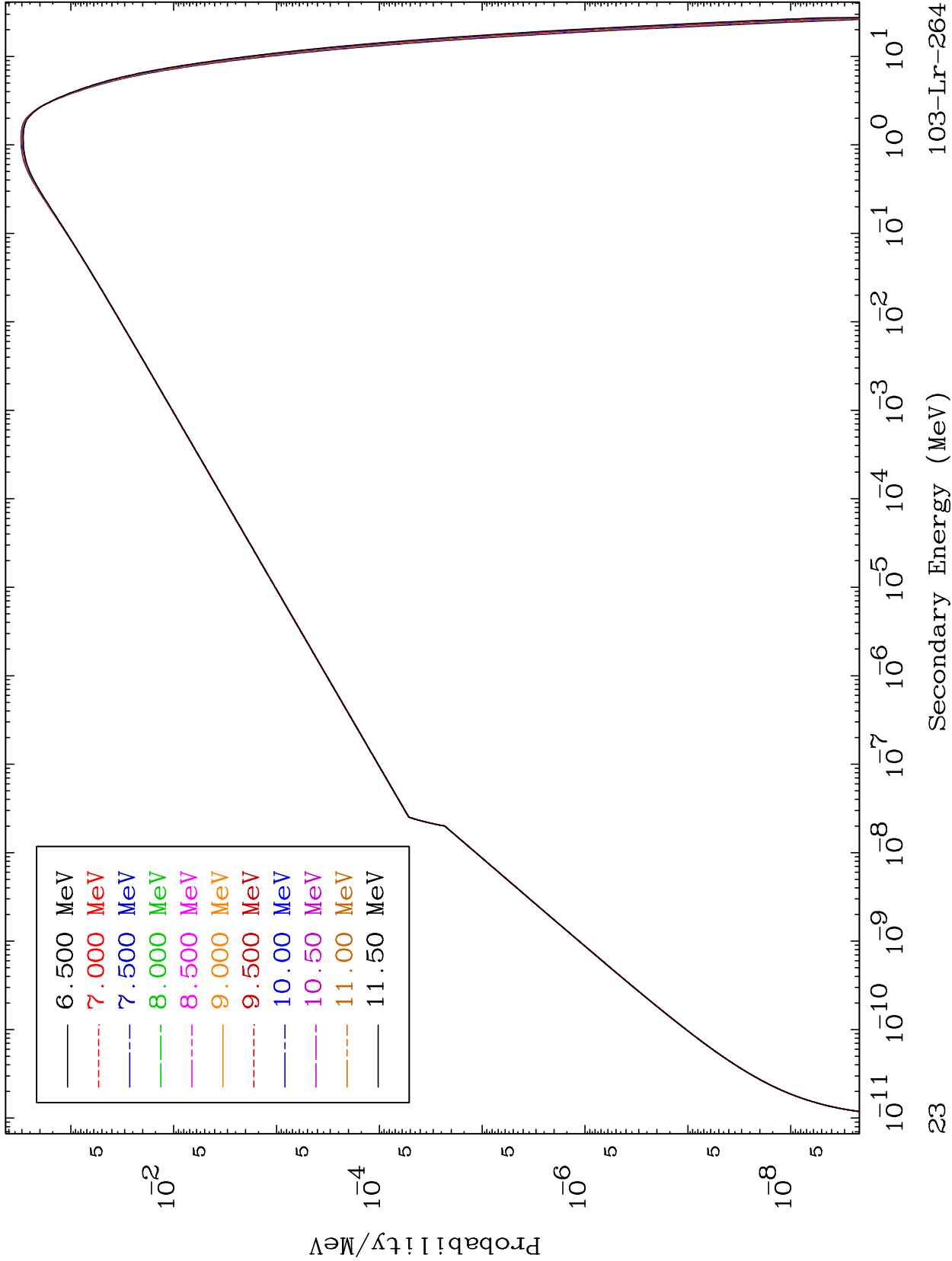
MAT 9993

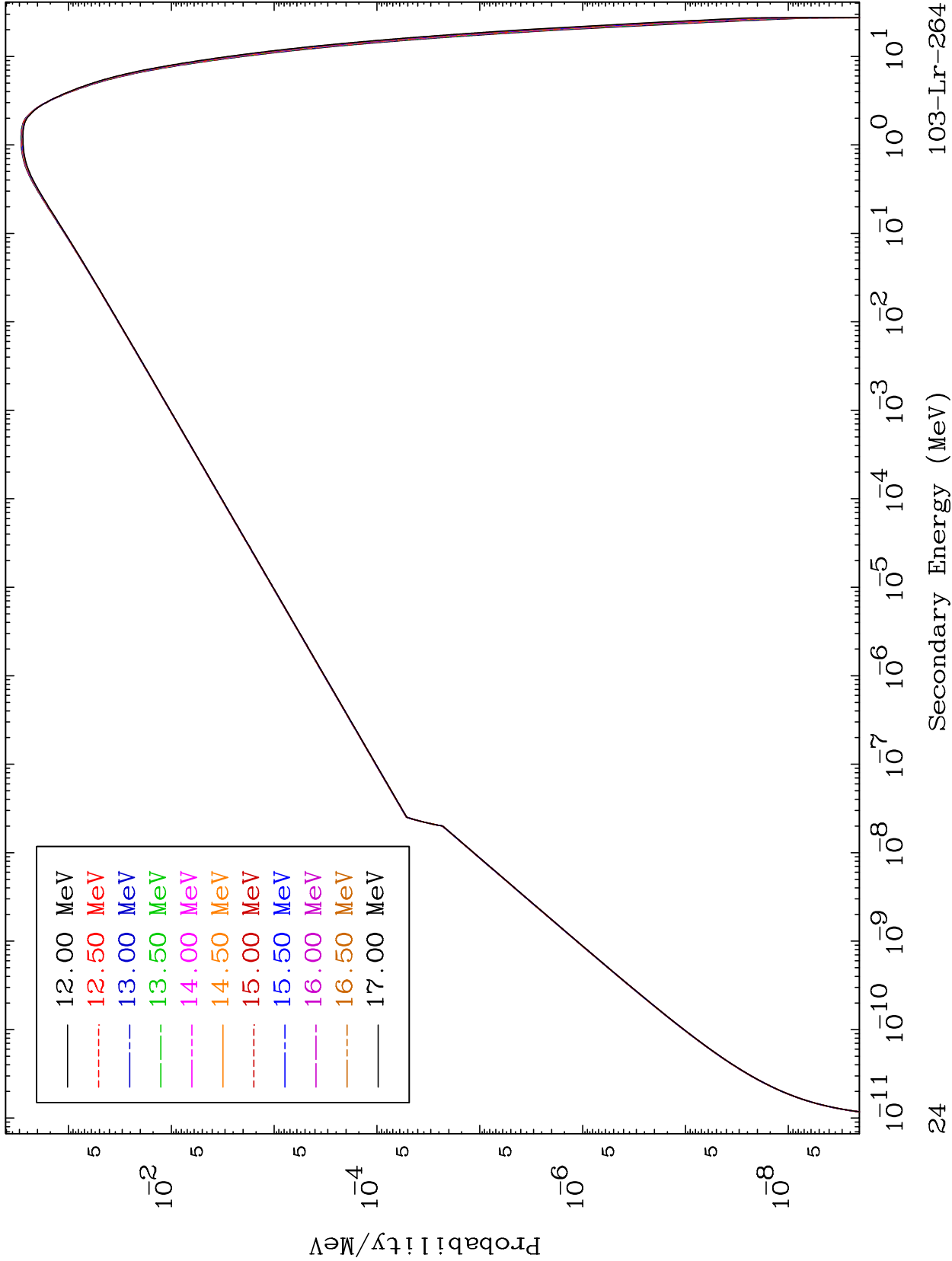
Fission Energy Distributions

103-Lr-264



Fission Energy Distributions

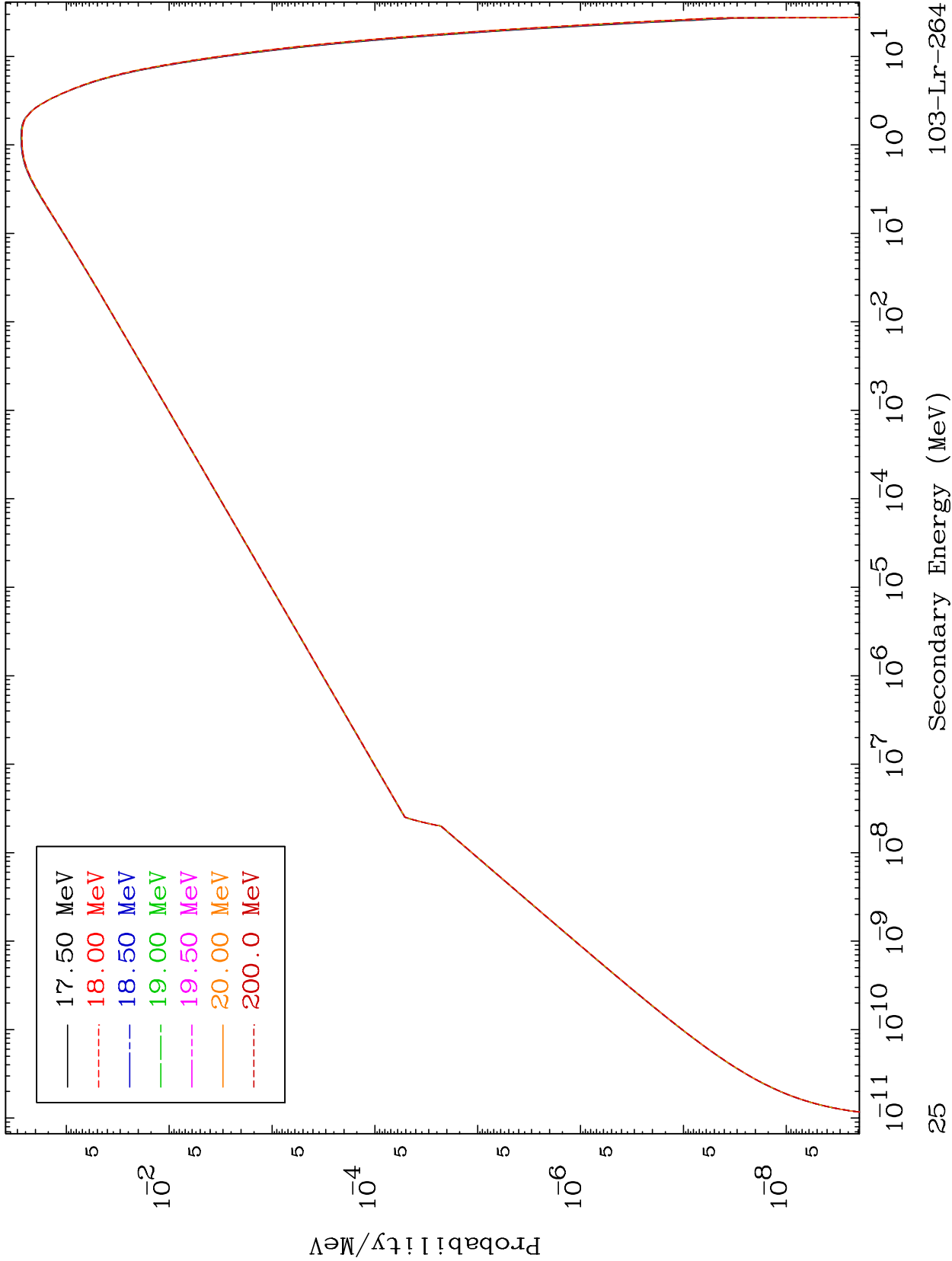




MAT 9993

Fission Energy Distributions

103-Lr-264

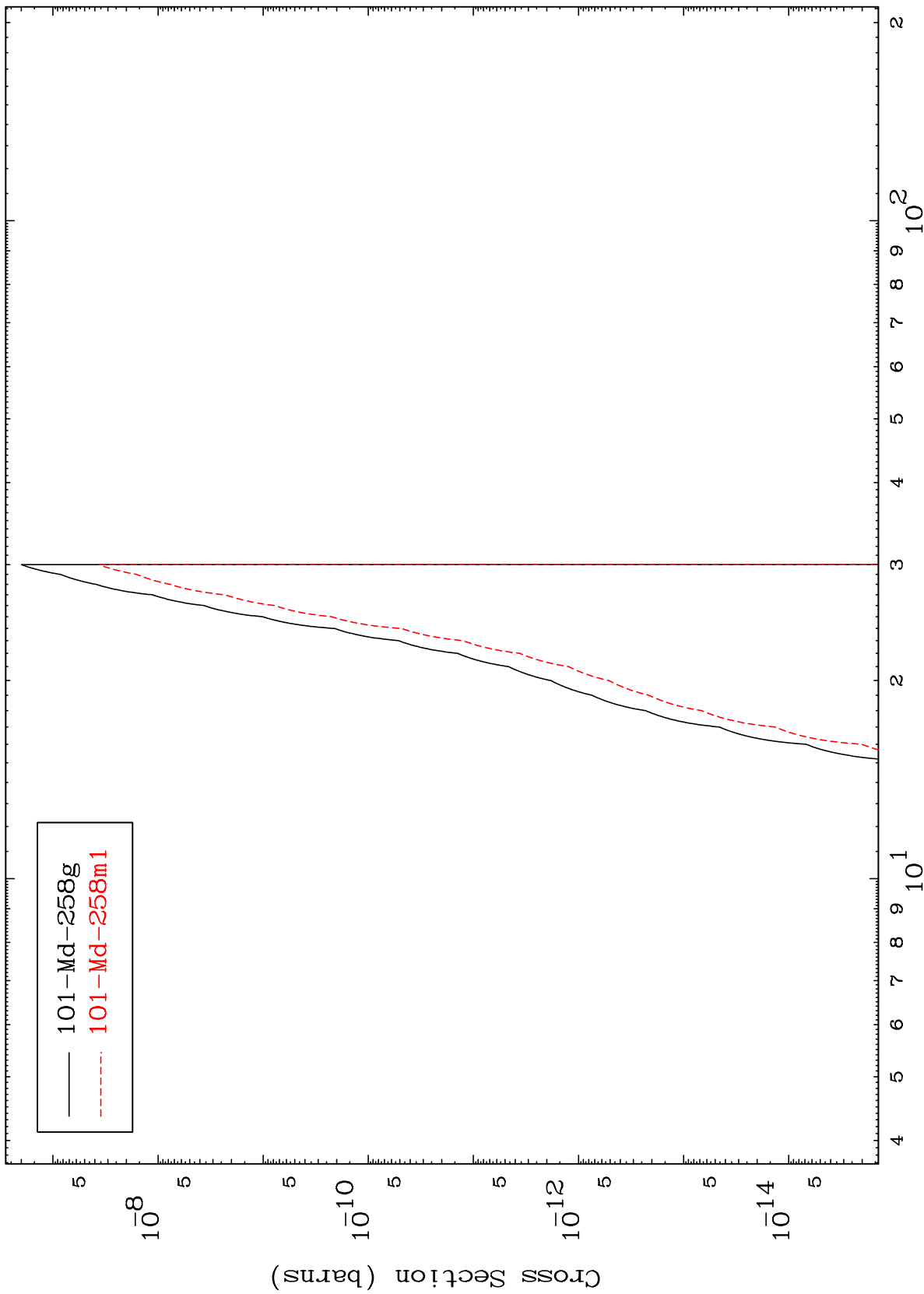


MAT 9993

$(n,3n) \alpha$

103-Lr-264

Radionuclide Production Cross Section



26

Incident Energy (MeV)

103-Lr-264