

Program EVALPLOT  
(Version 2021-1)

by

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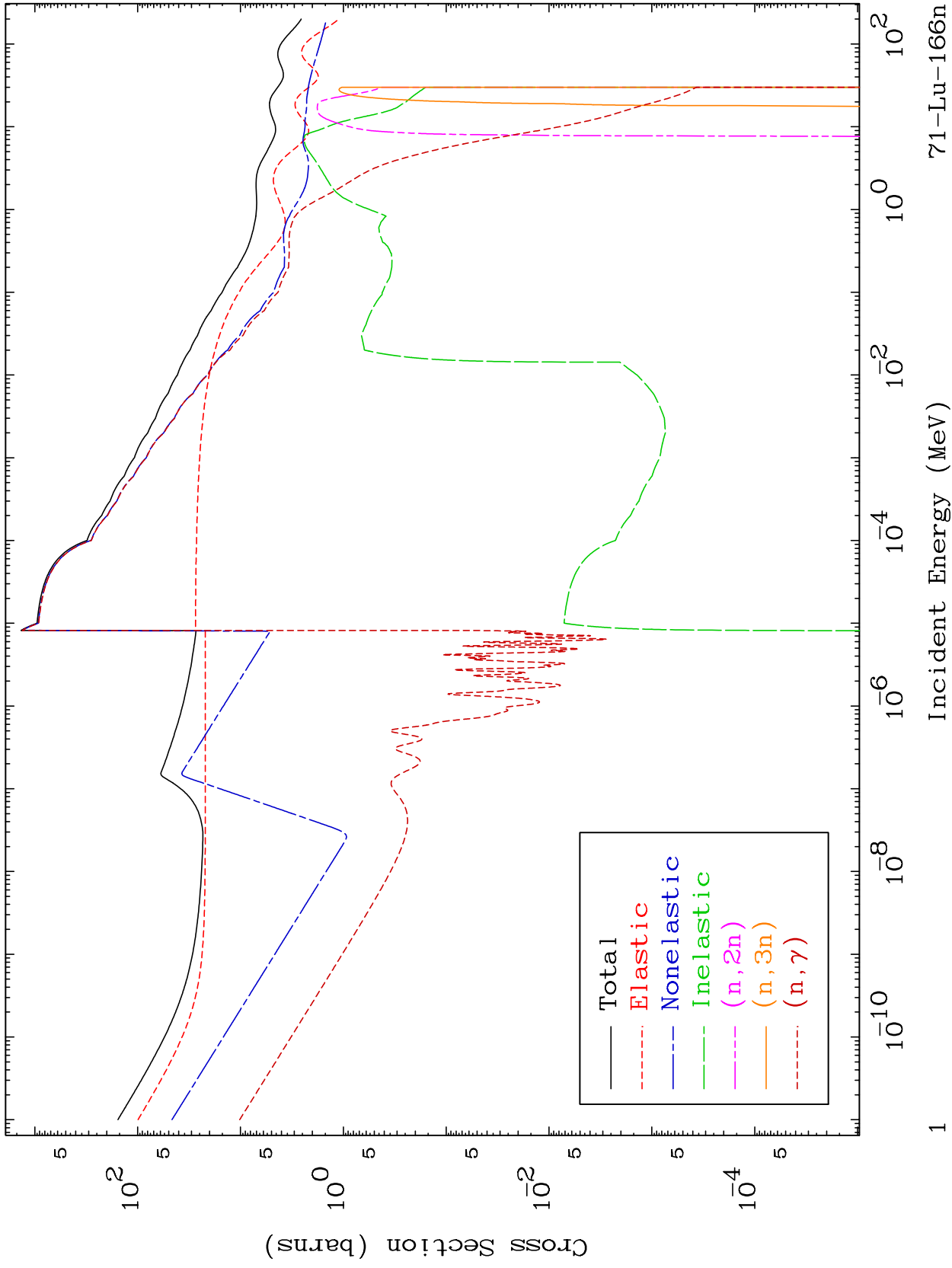
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 7100

Neutron Major  
293 Kelvin Cross Sections

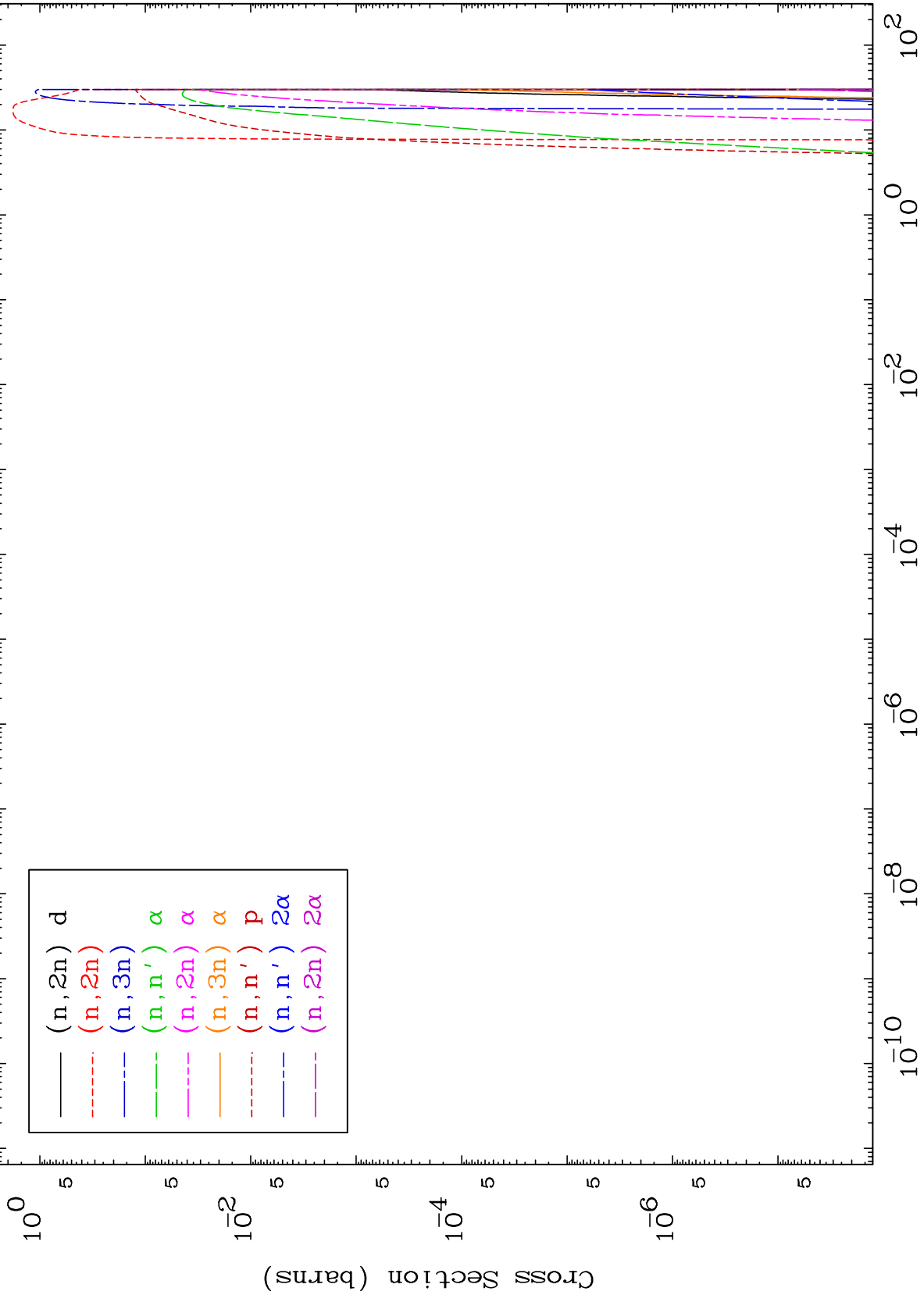
71-Lu-166n



MAT 7100

Neutron Absorption  
293 Kelvin Cross Sections

71-Lu-166n



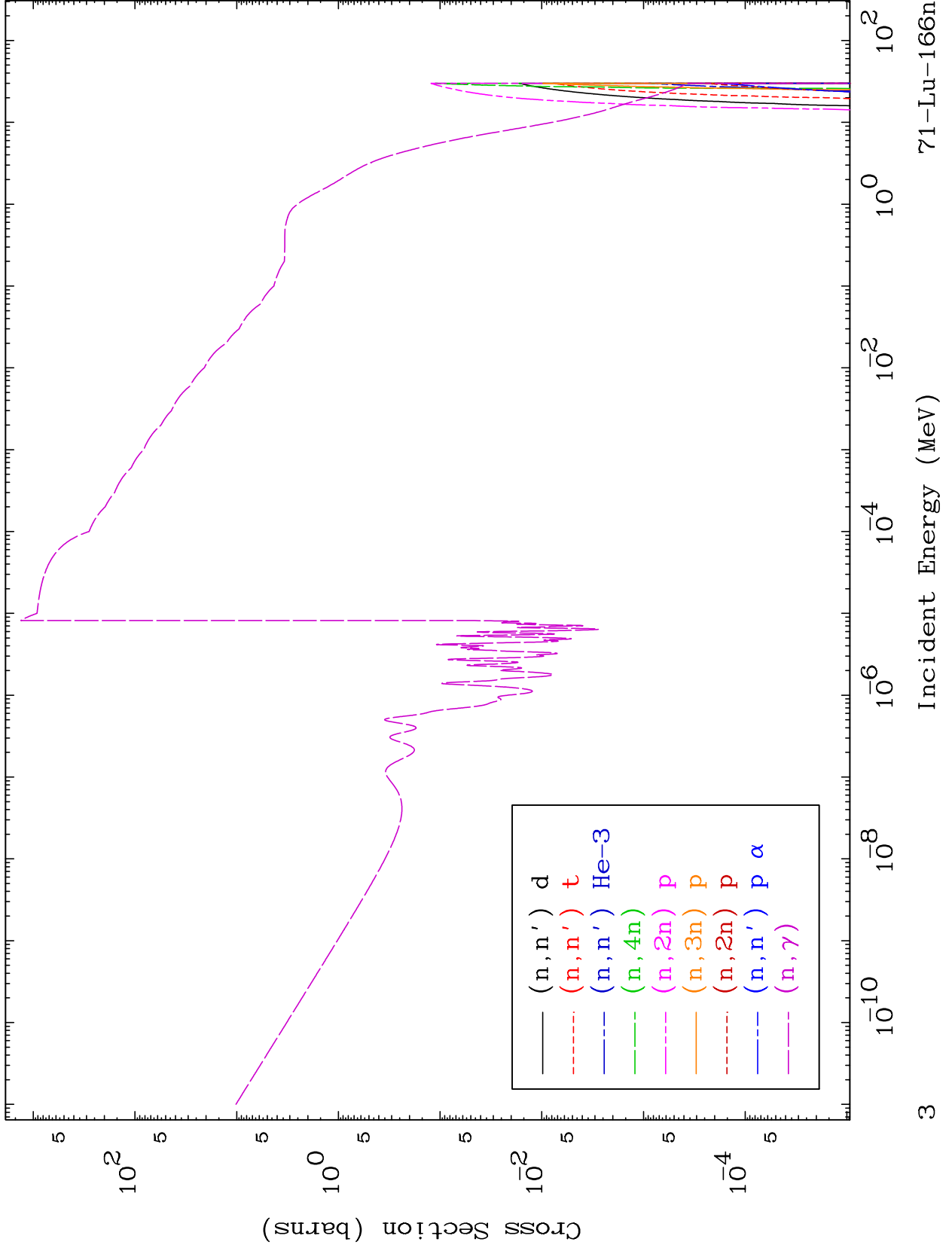
71-Lu-166n

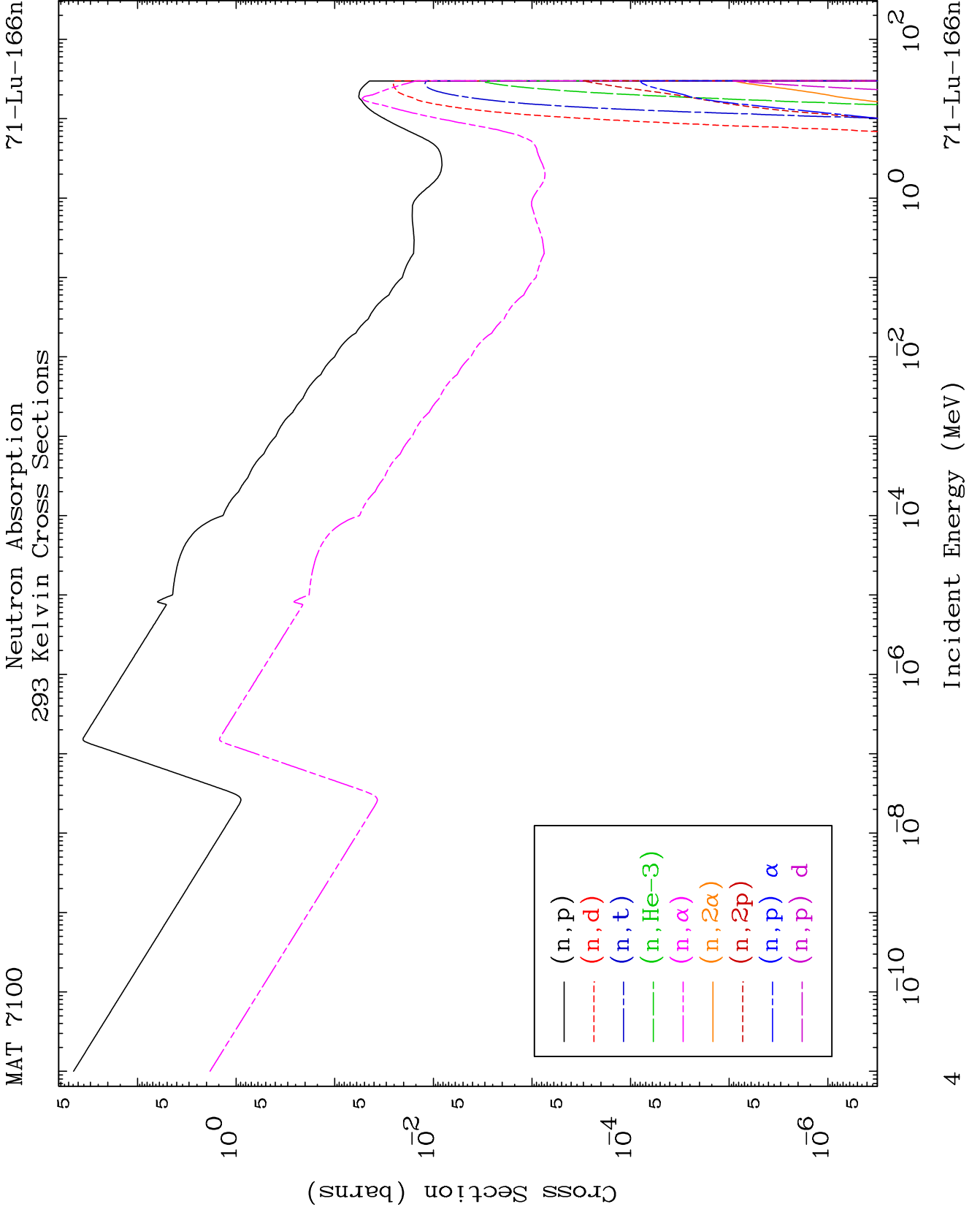
Incident Energy (MeV)

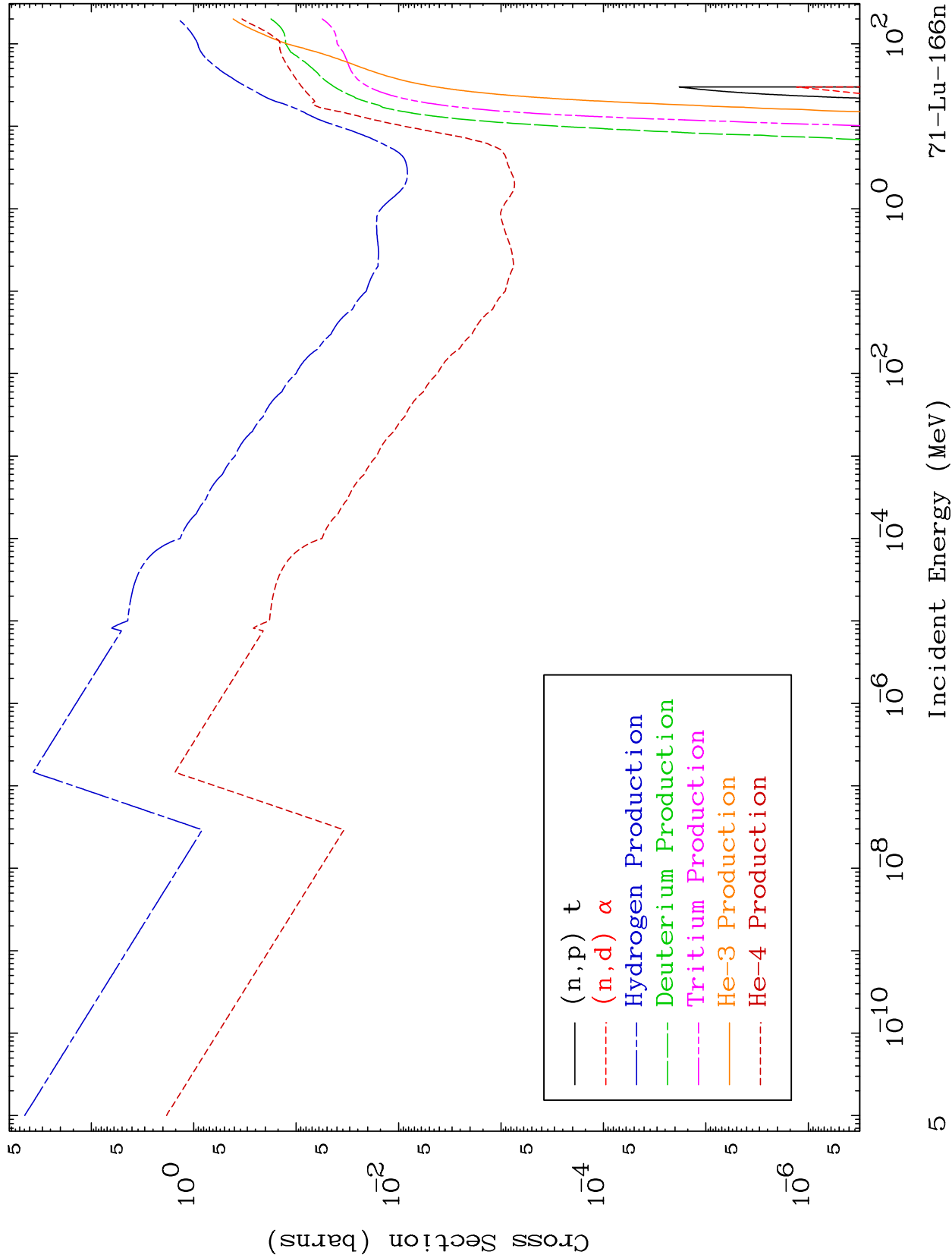
MAT 7100

Neutron Absorption  
293 Kelvin Cross Sections

71-Lu-166n



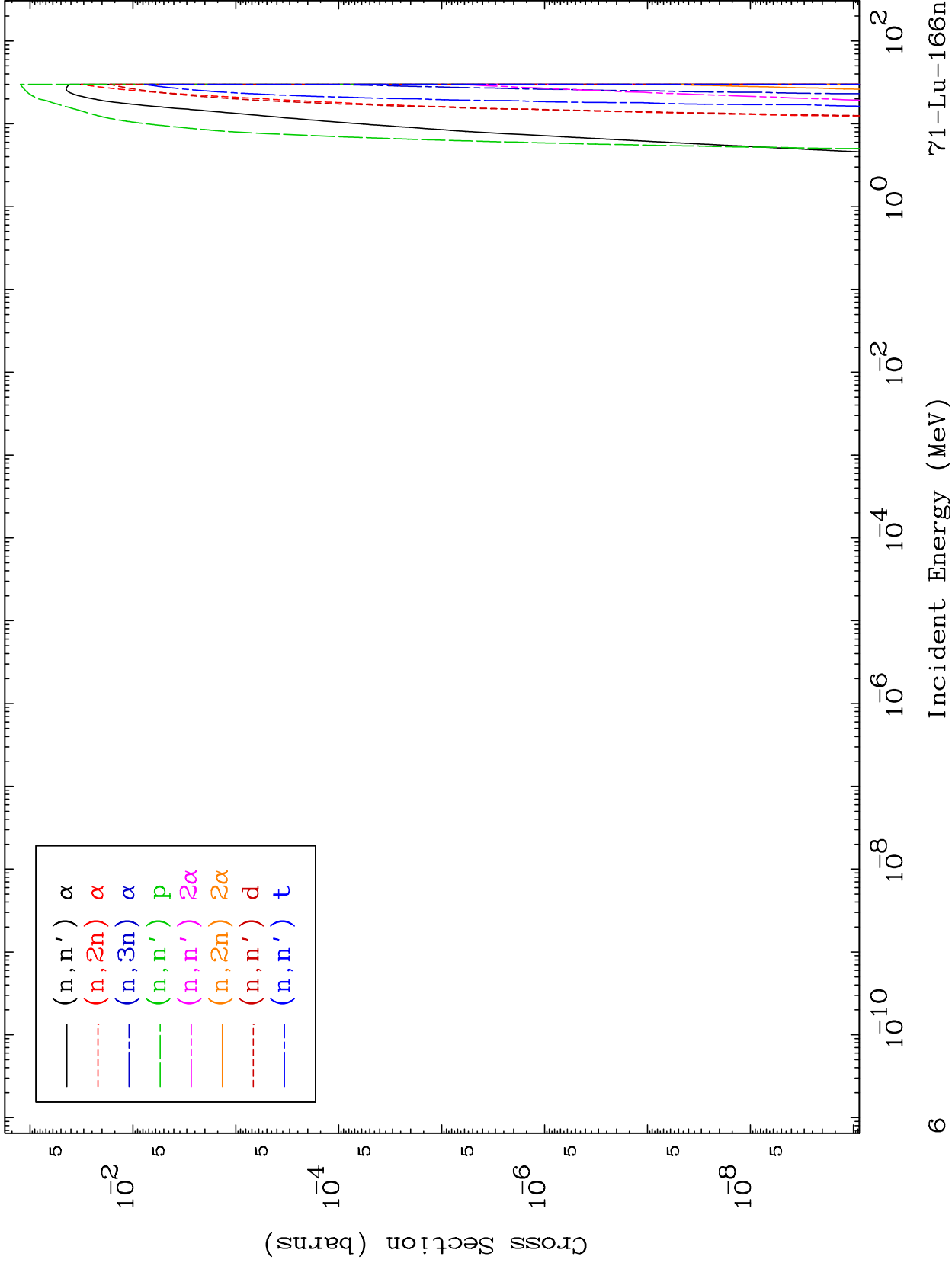




MAT 7100

Charged Particle  
293 Kelvin Cross Sections

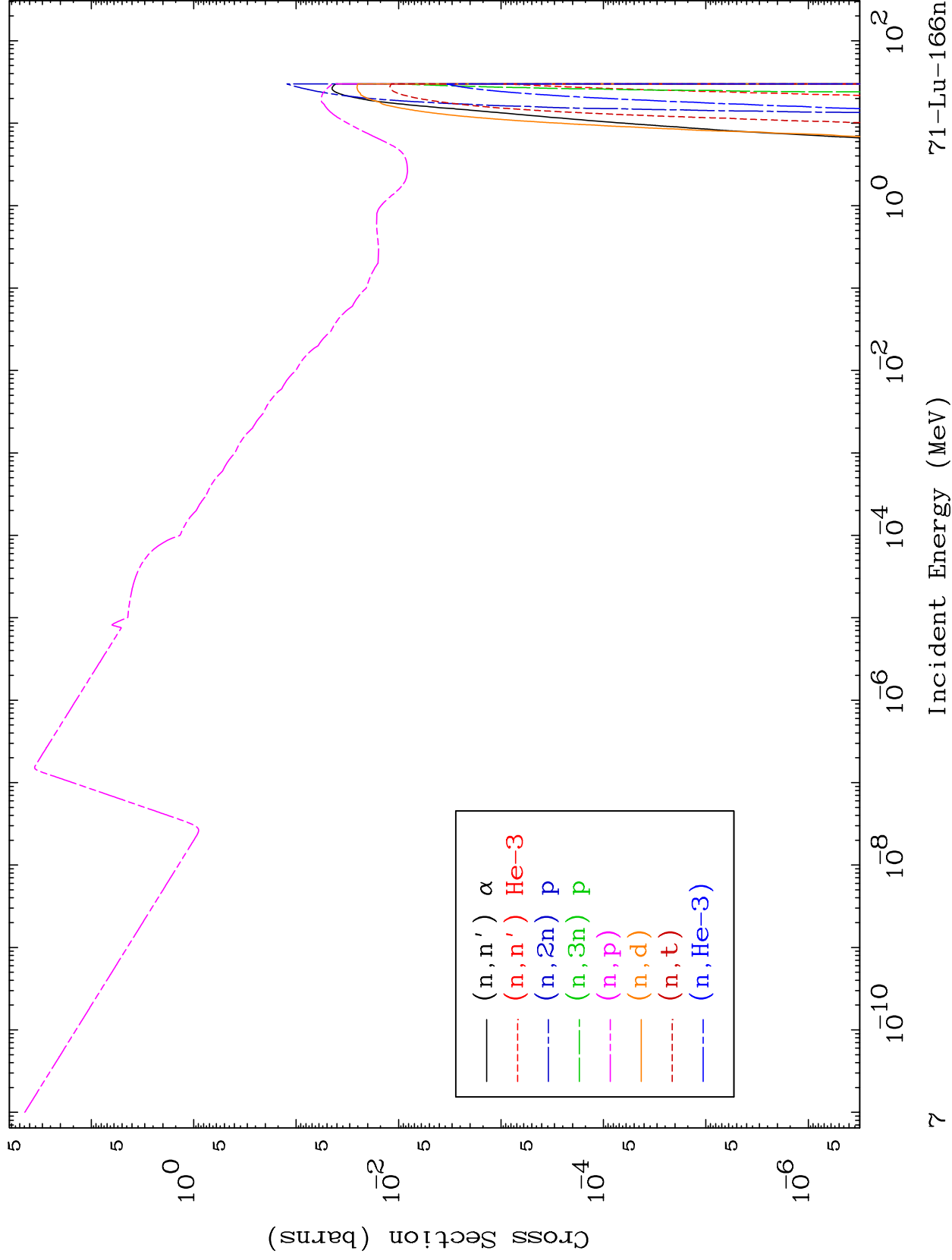
<sup>71</sup>Lu-166n



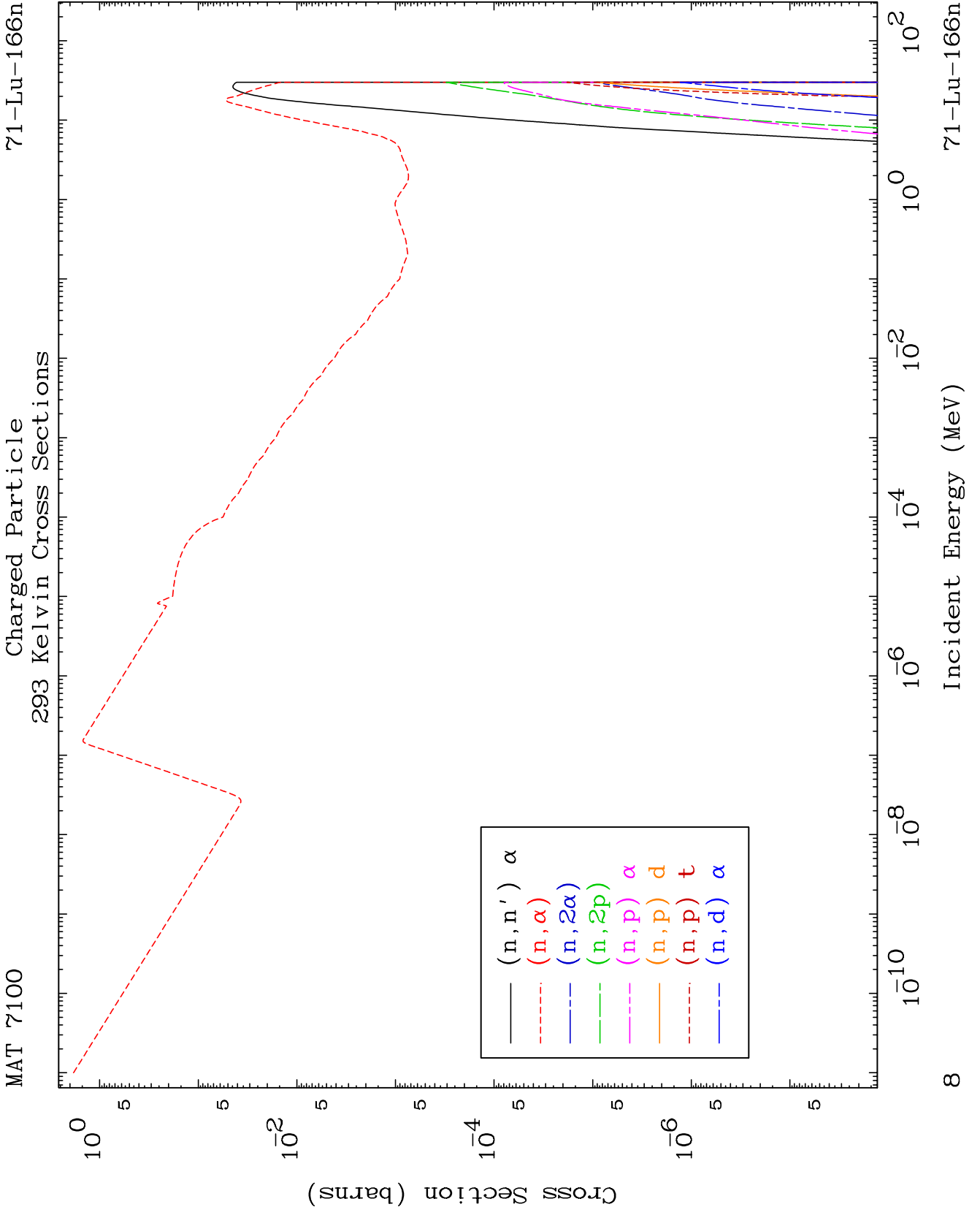
MAT 7100

Charged Particle  
293 Kelvin Cross Sections

71-Lu-166n



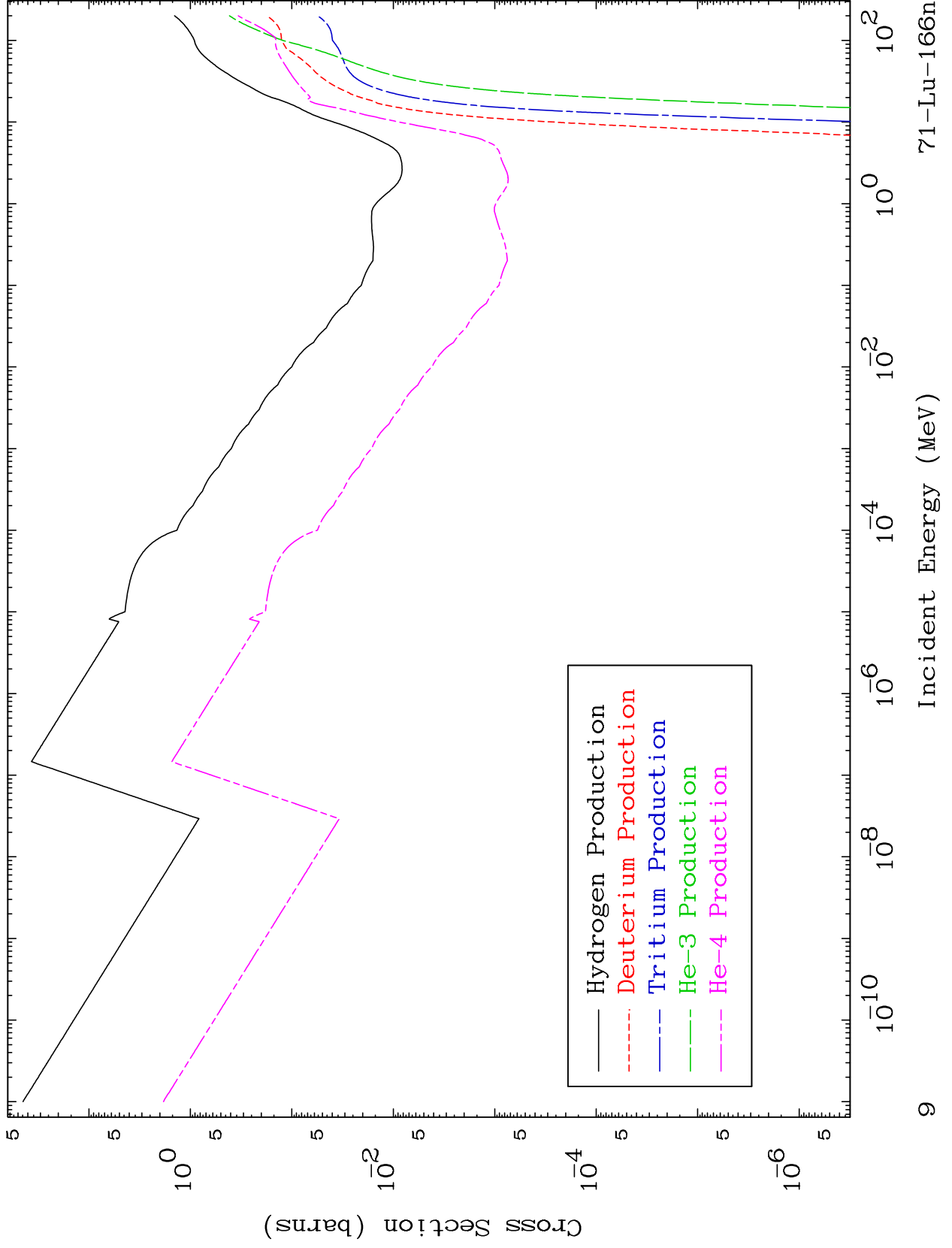




MAT 7100

Particle Production  
293 Kelvin Cross Sections

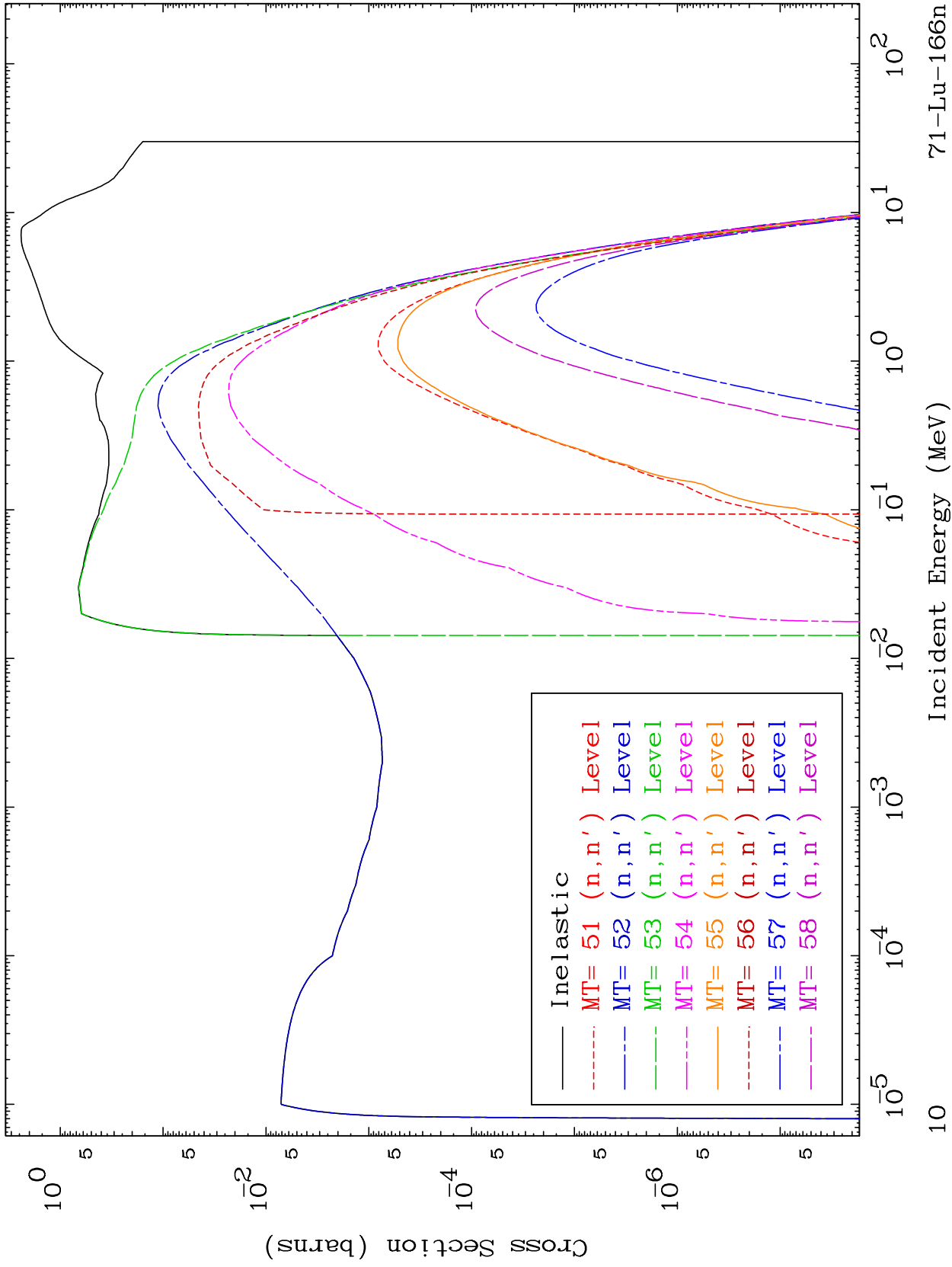
<sup>71</sup>Lu-166n



MAT 7100

(n,n') Levels  
293 Kelvin Cross Sections

71-Lu-166n



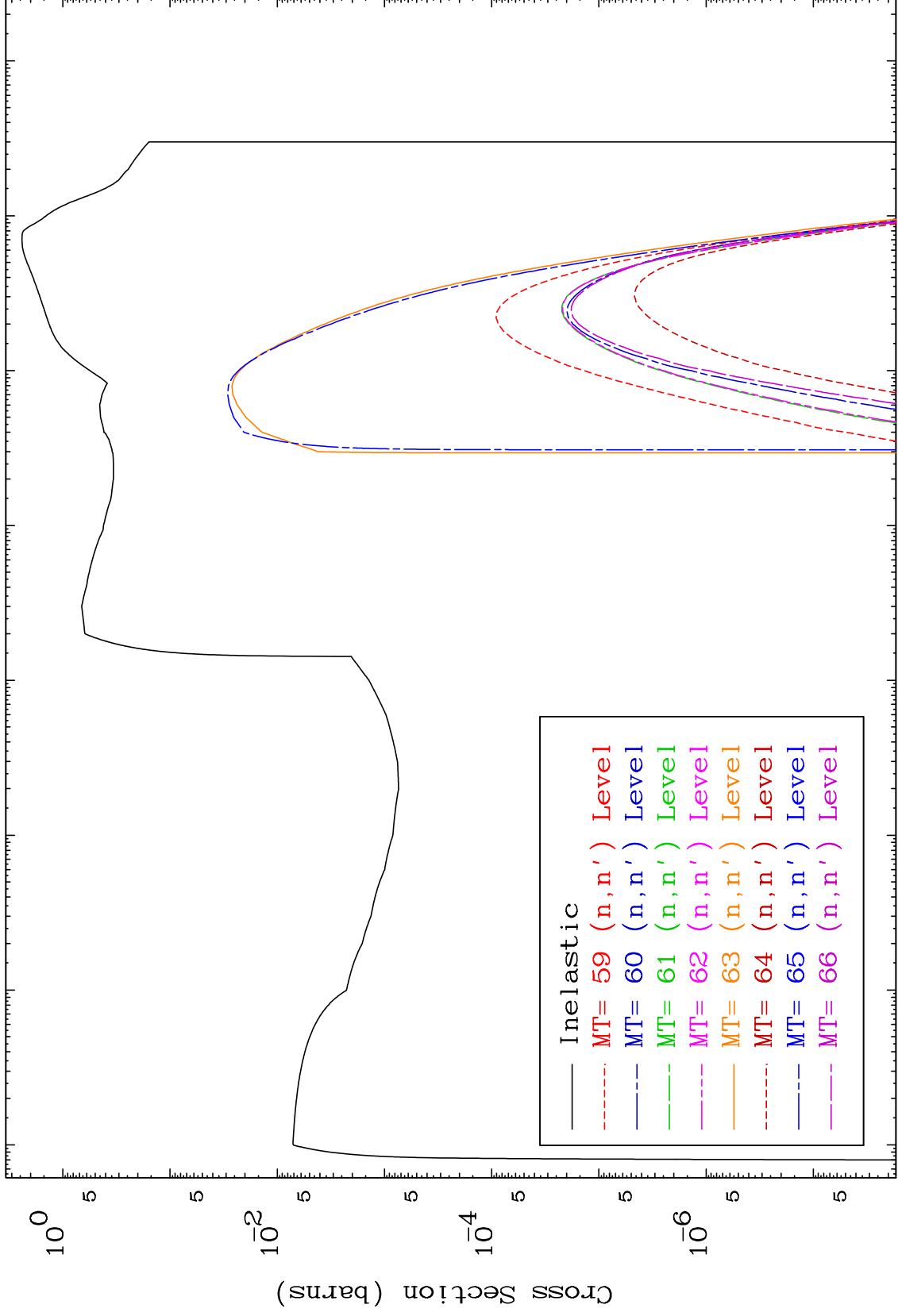
71-Lu-166n

Incident Energy (MeV)

MAT 7100

(n,n') Levels  
293 Kelvin Cross Sections

71-Lu-166n



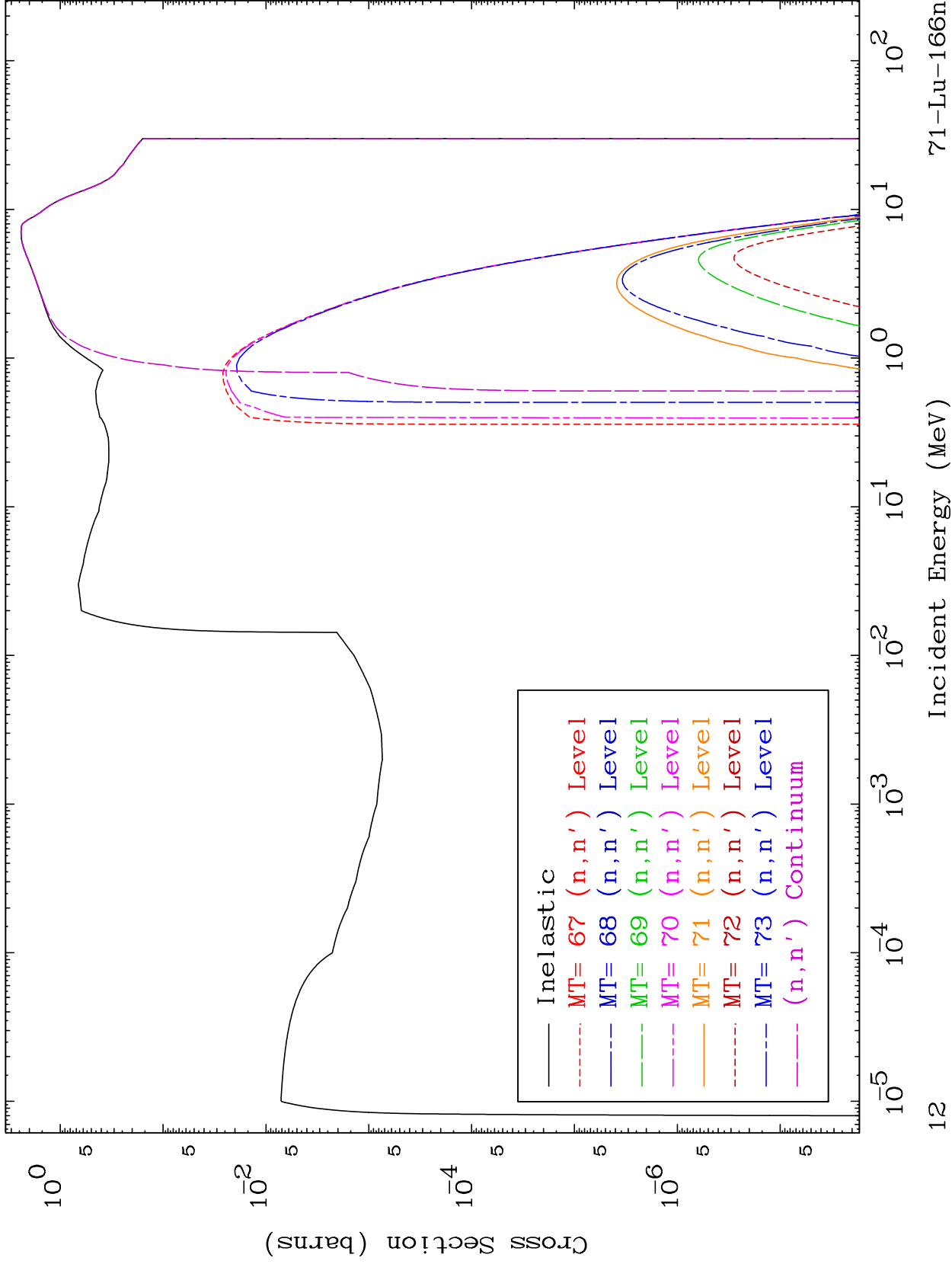
71-Lu-166n

Incident Energy (MeV)

MAT 7100

(n,n') Levels  
293 Kelvin Cross Sections

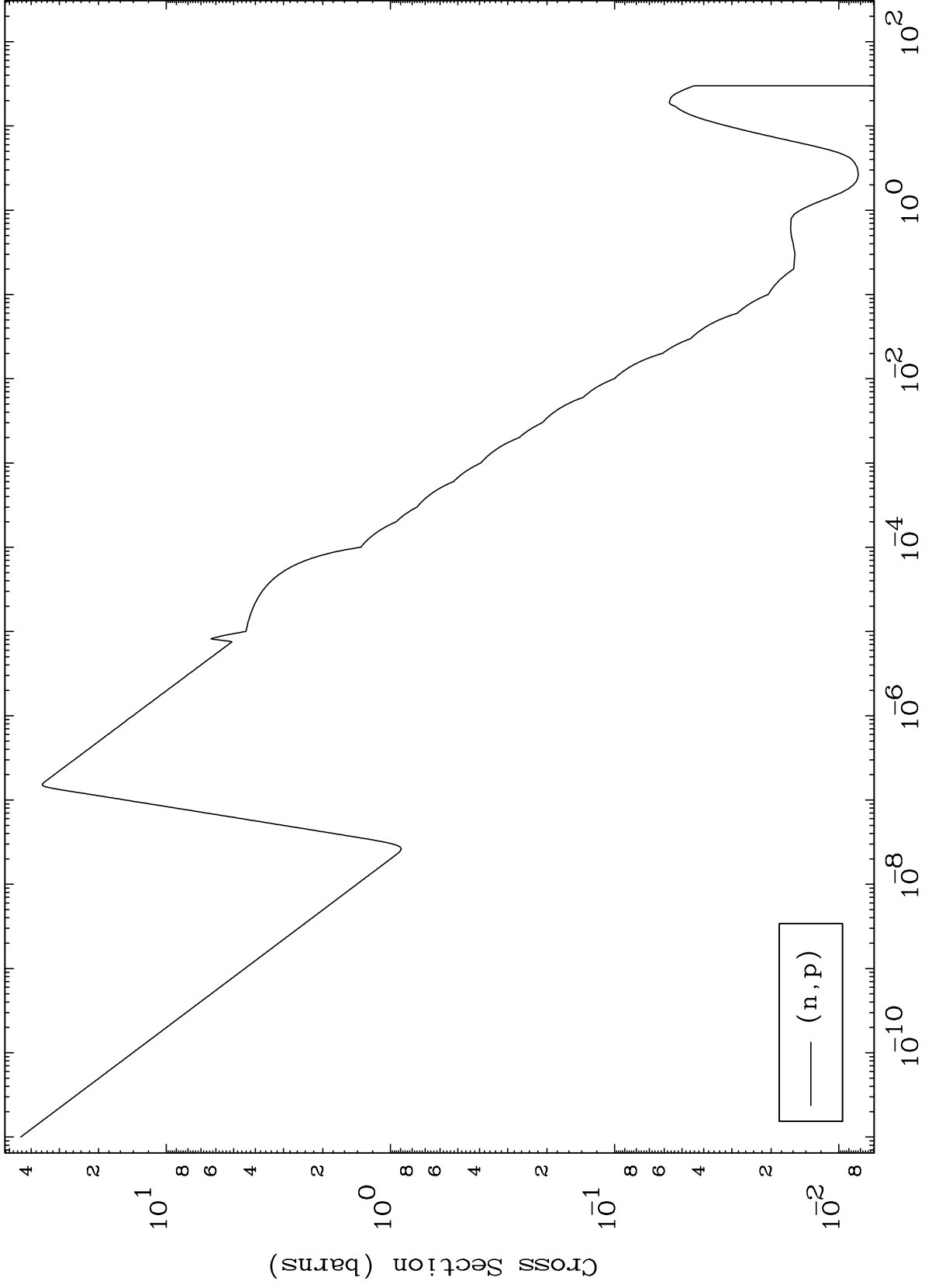
71-Lu-166n



MAT 7100

(n,p) Levels  
293 Kelvin Cross Sections

71-Lu-166n



13

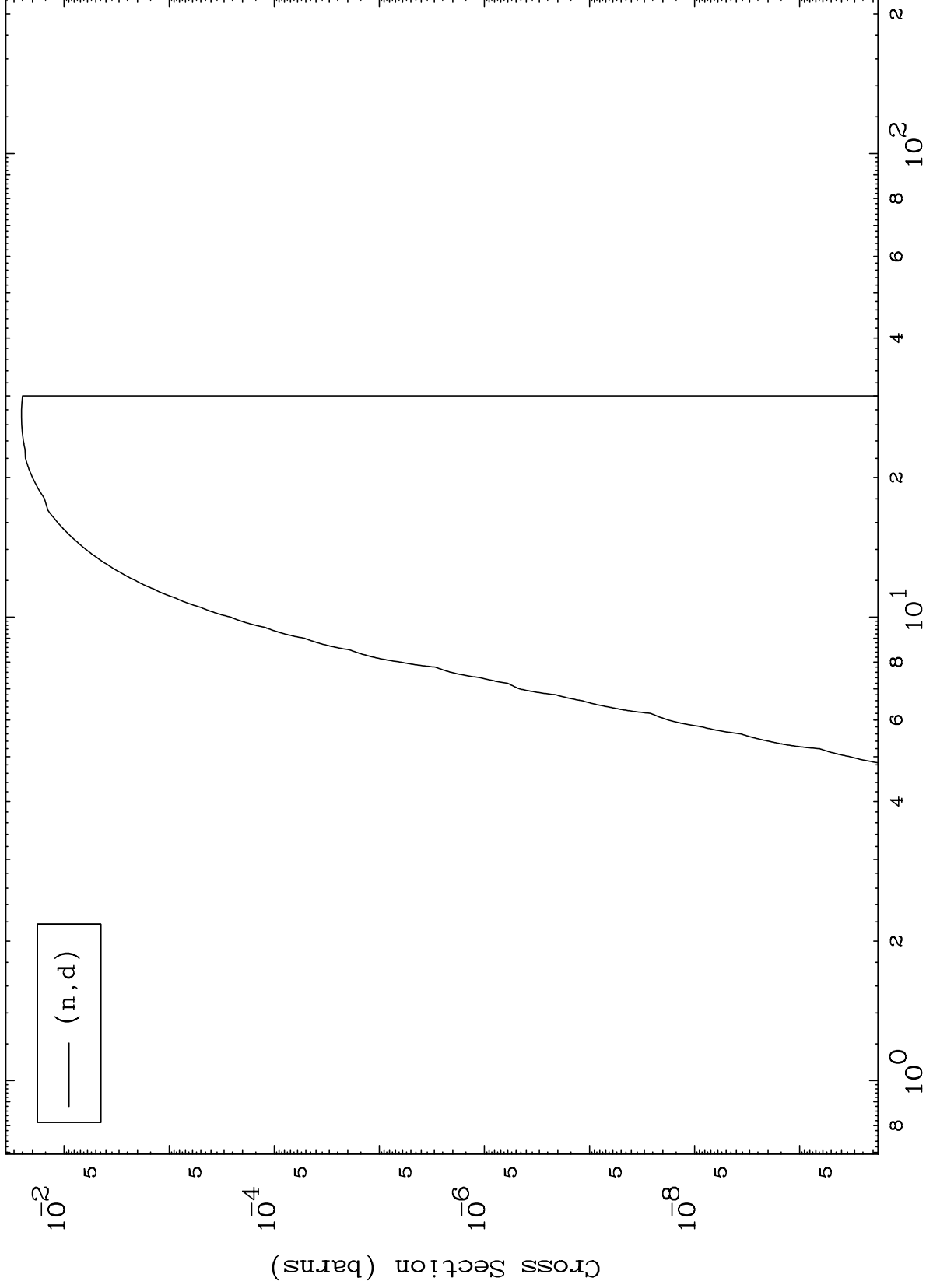
Incident Energy (MeV)

71-Lu-166n

MAT 7100

(n,d) Levels  
293 Kelvin Cross Sections

71-Lu-166n



14

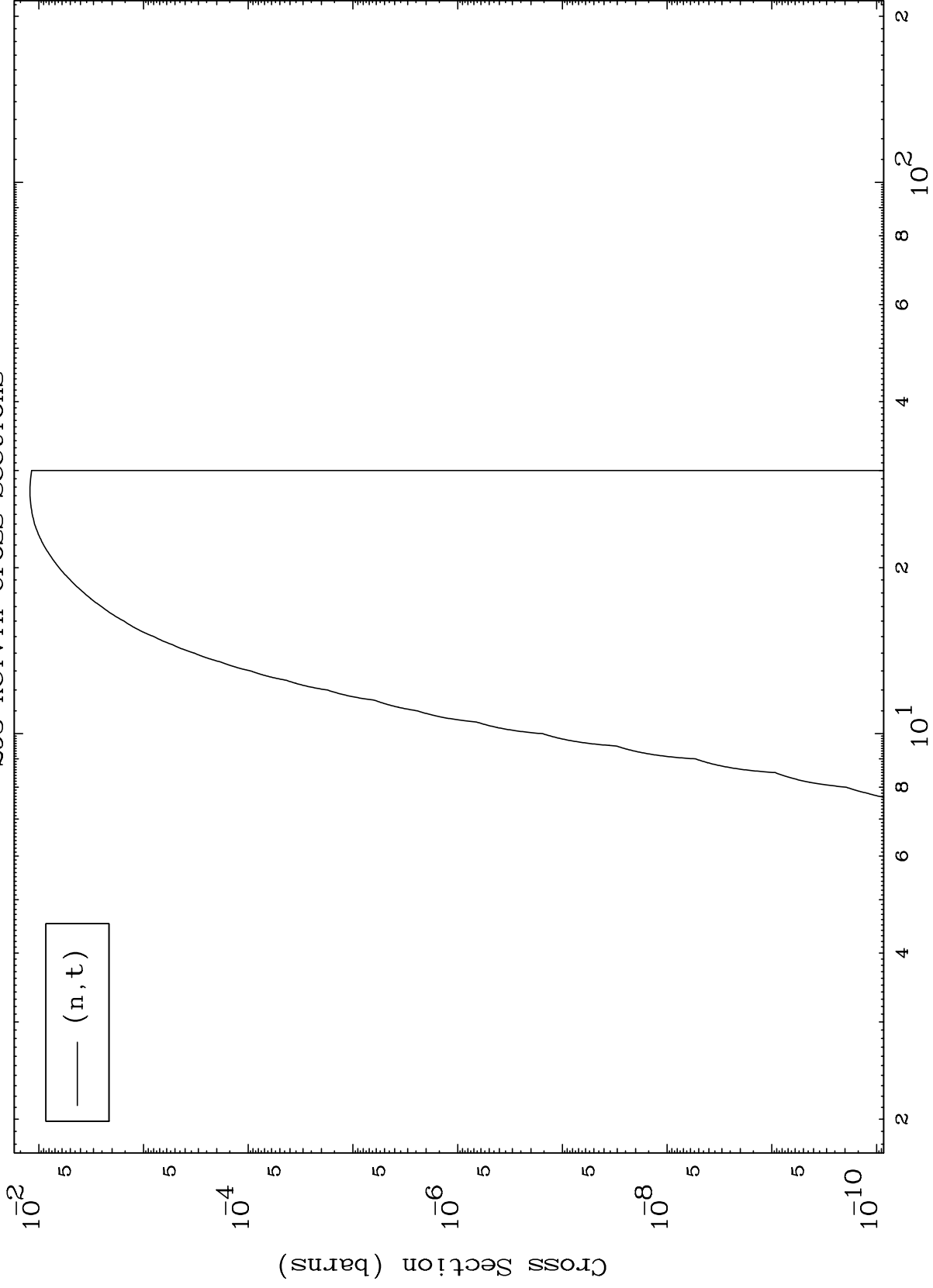
Incident Energy (MeV)

71-Lu-166n

MAT 7100

(n,t) Levels  
293 Kelvin Cross Sections

71-Lu-166n



15

Incident Energy (MeV)

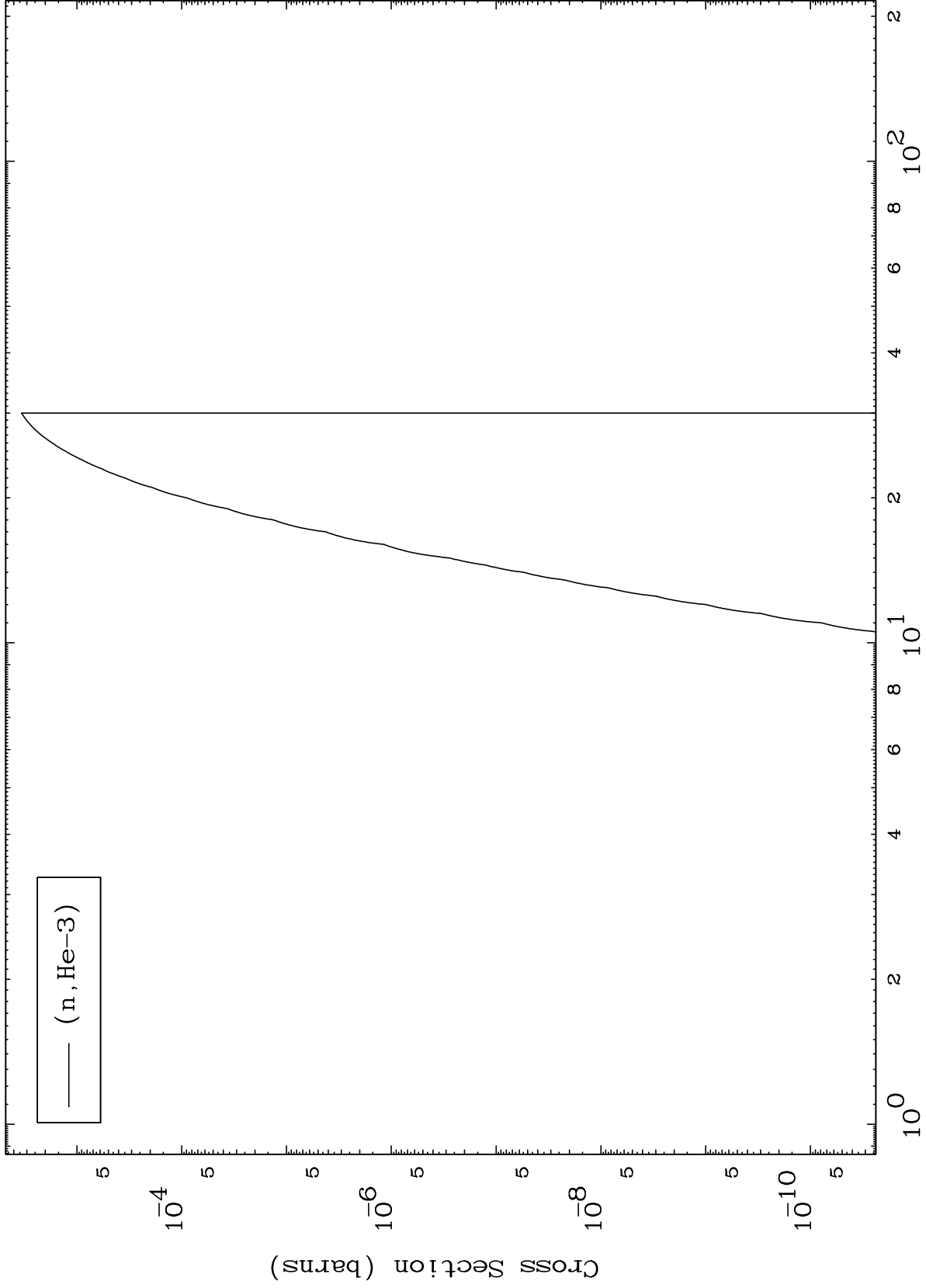
71-Lu-166n



MAT 7100

(n,He3) Levels  
293 Kelvin Cross Sections

71-Lu-166n



16

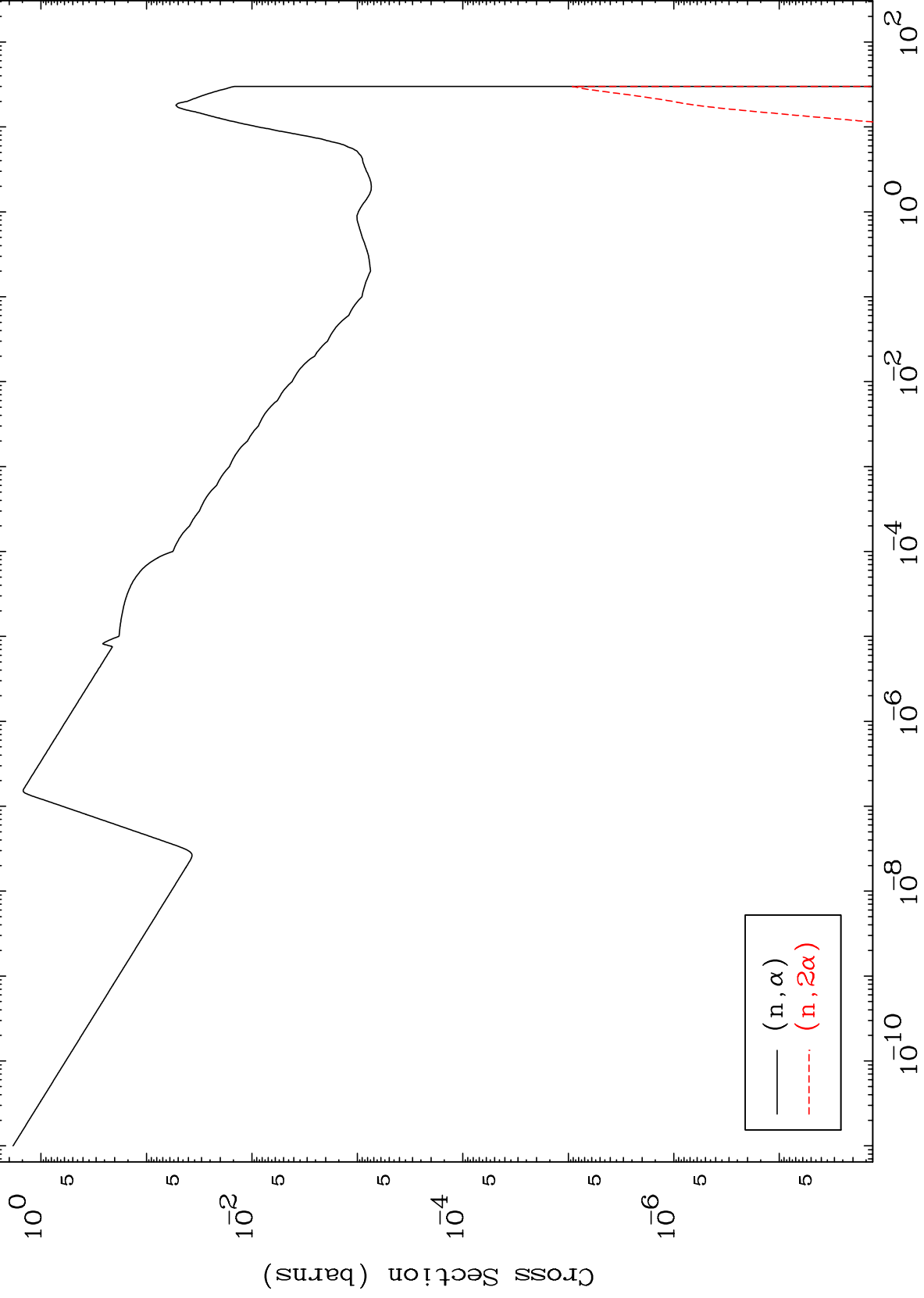
Incident Energy (MeV)

71-Lu-166n

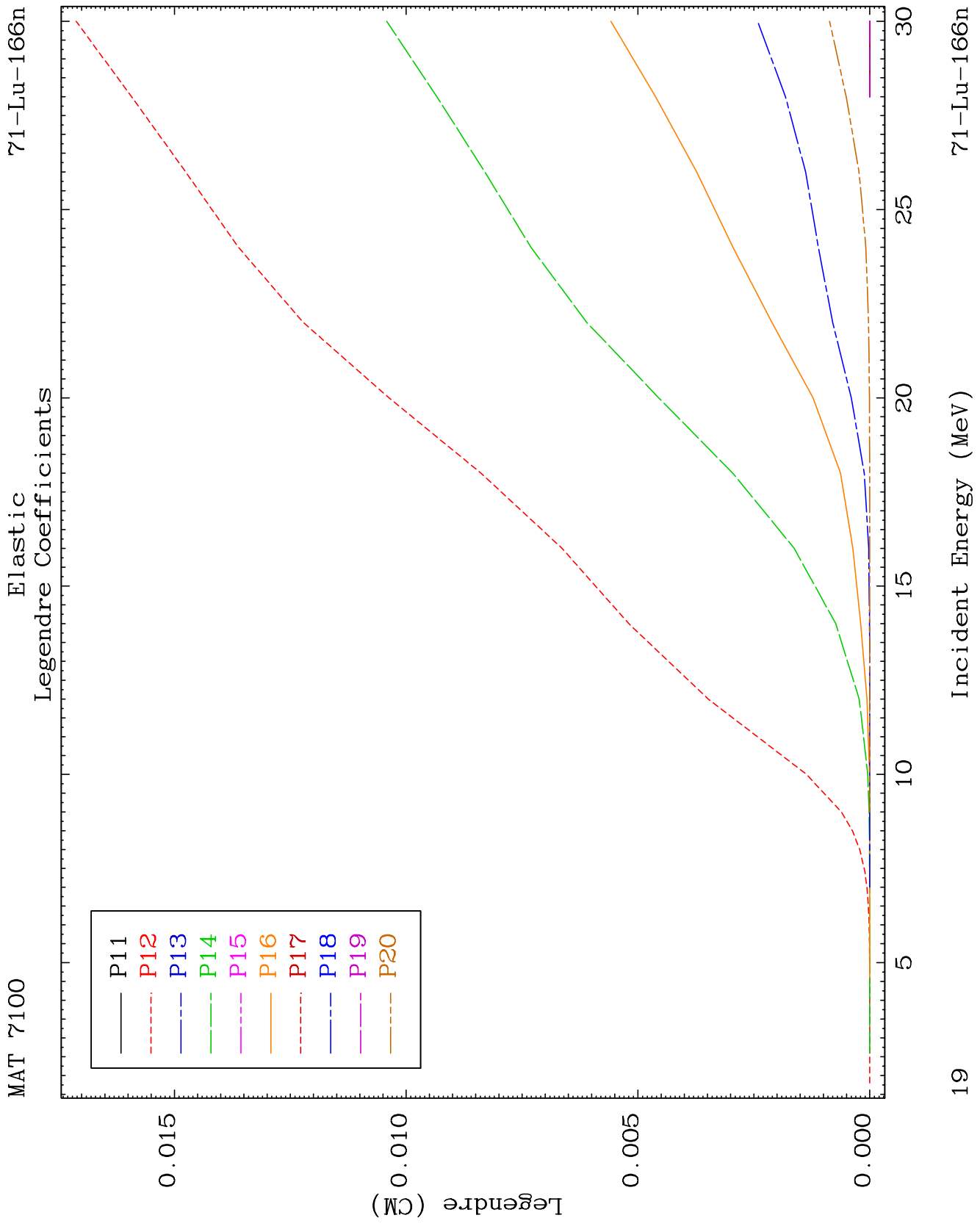
MAT 7100

(n,α) Levels  
293 Kelvin Cross Sections

<sup>71</sup>Lu-166n





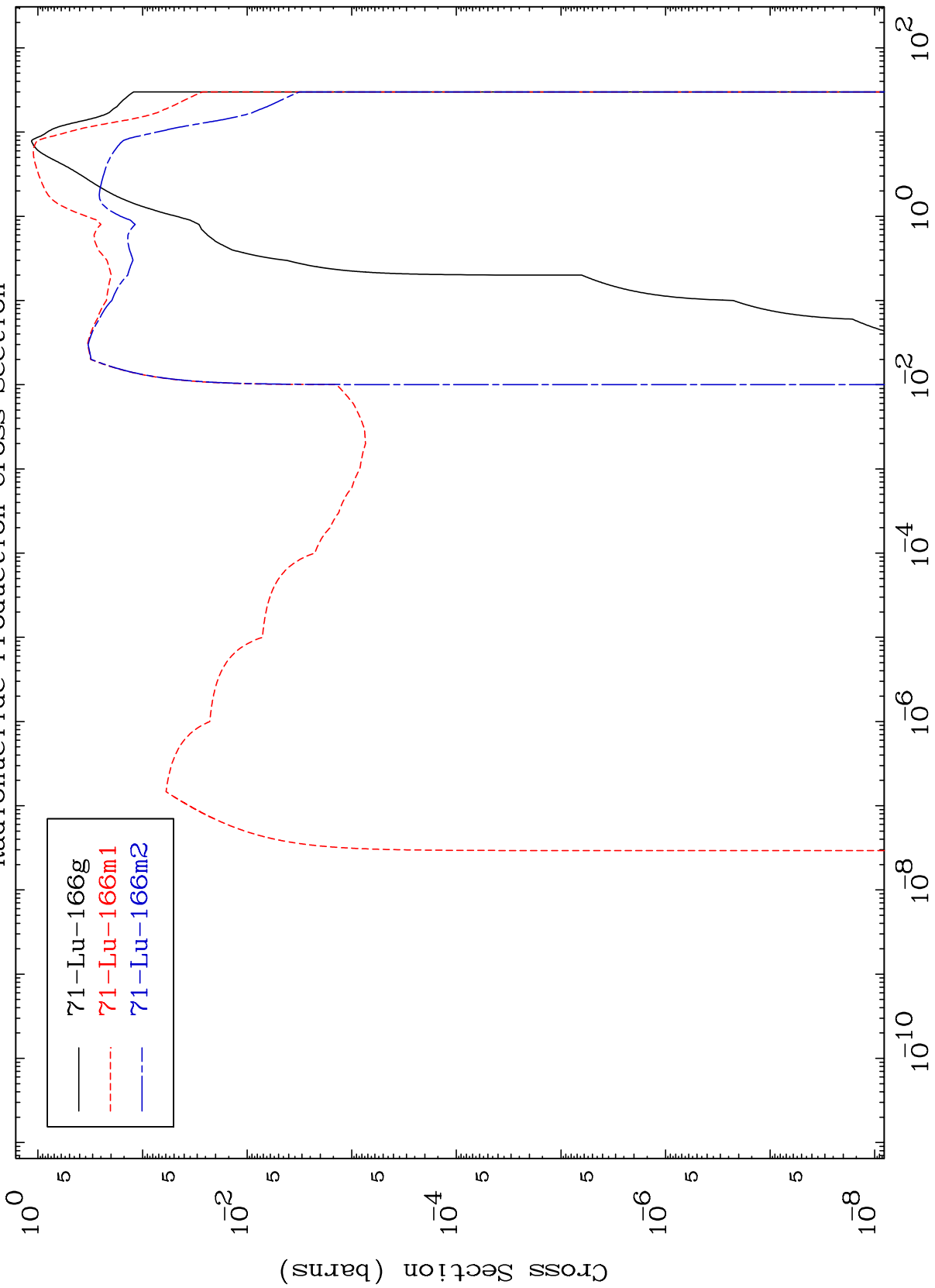




MAT 7100

<sup>71</sup>Lu-166n

Inelastic  
Radionuclide Production Cross Section



<sup>71</sup>Lu-166n

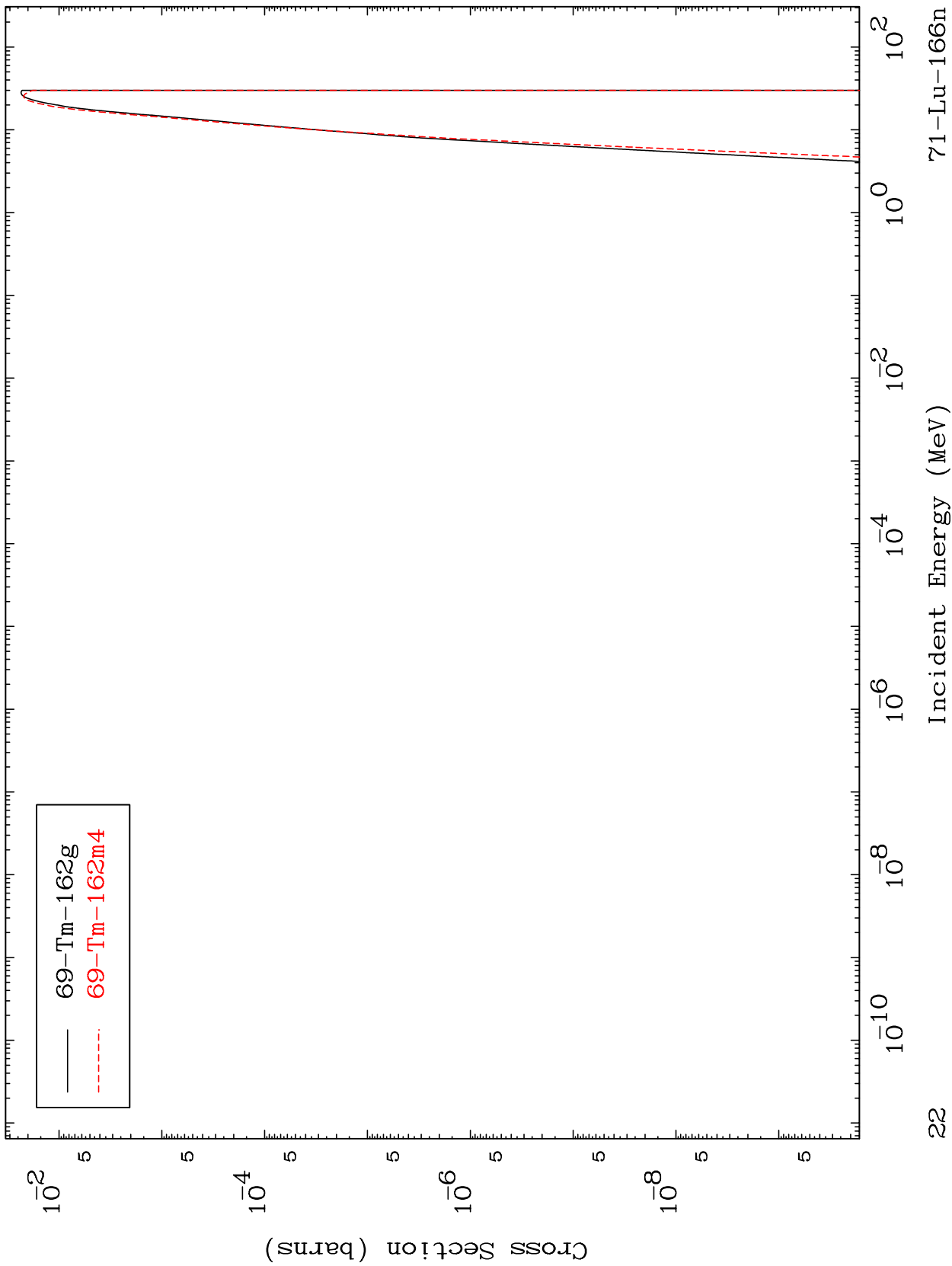
Incident Energy (MeV)

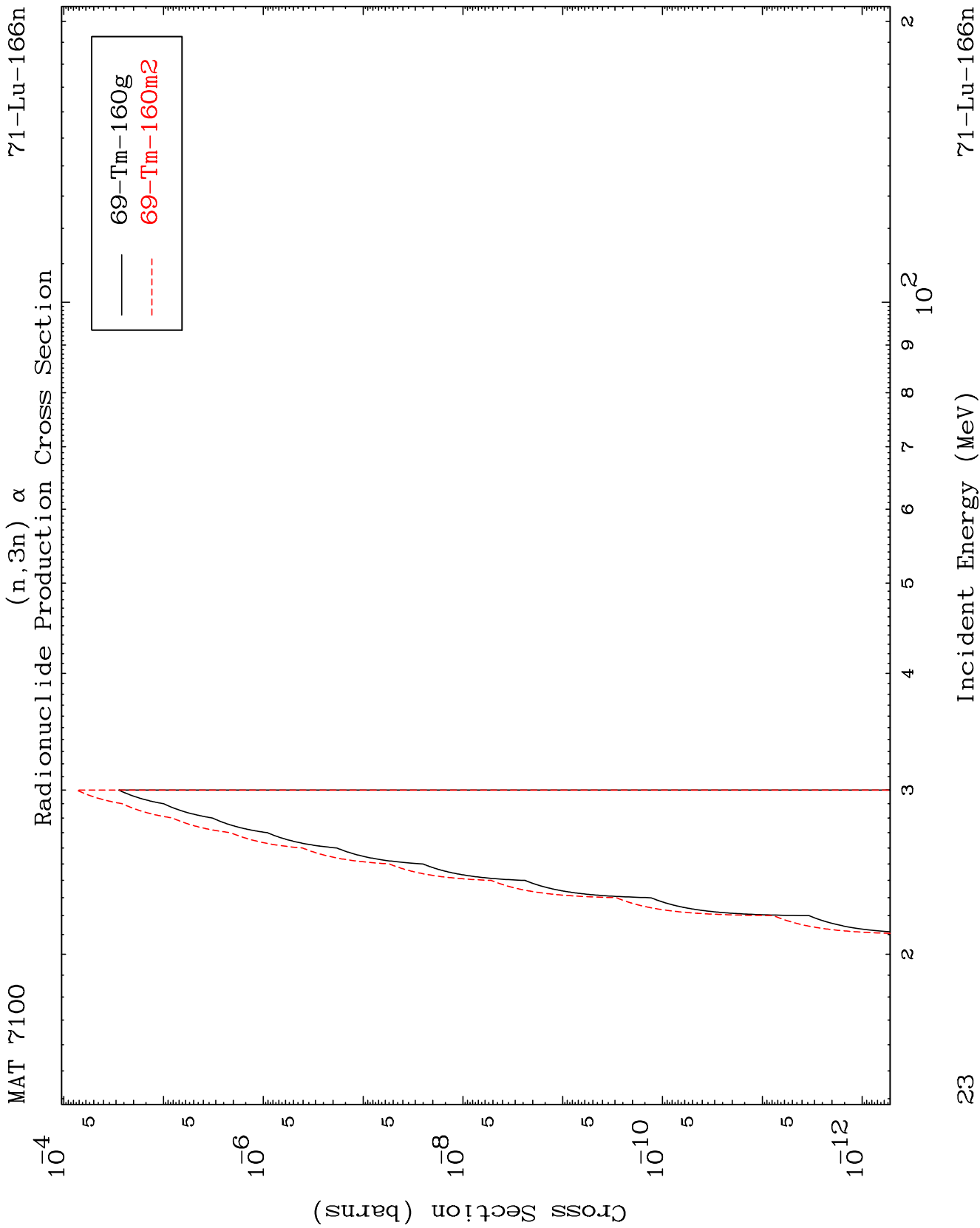
MAT 7100

$(n, n') \alpha$

$^{71}\text{Lu}-166\text{n}$

Radionuclide Production Cross Section





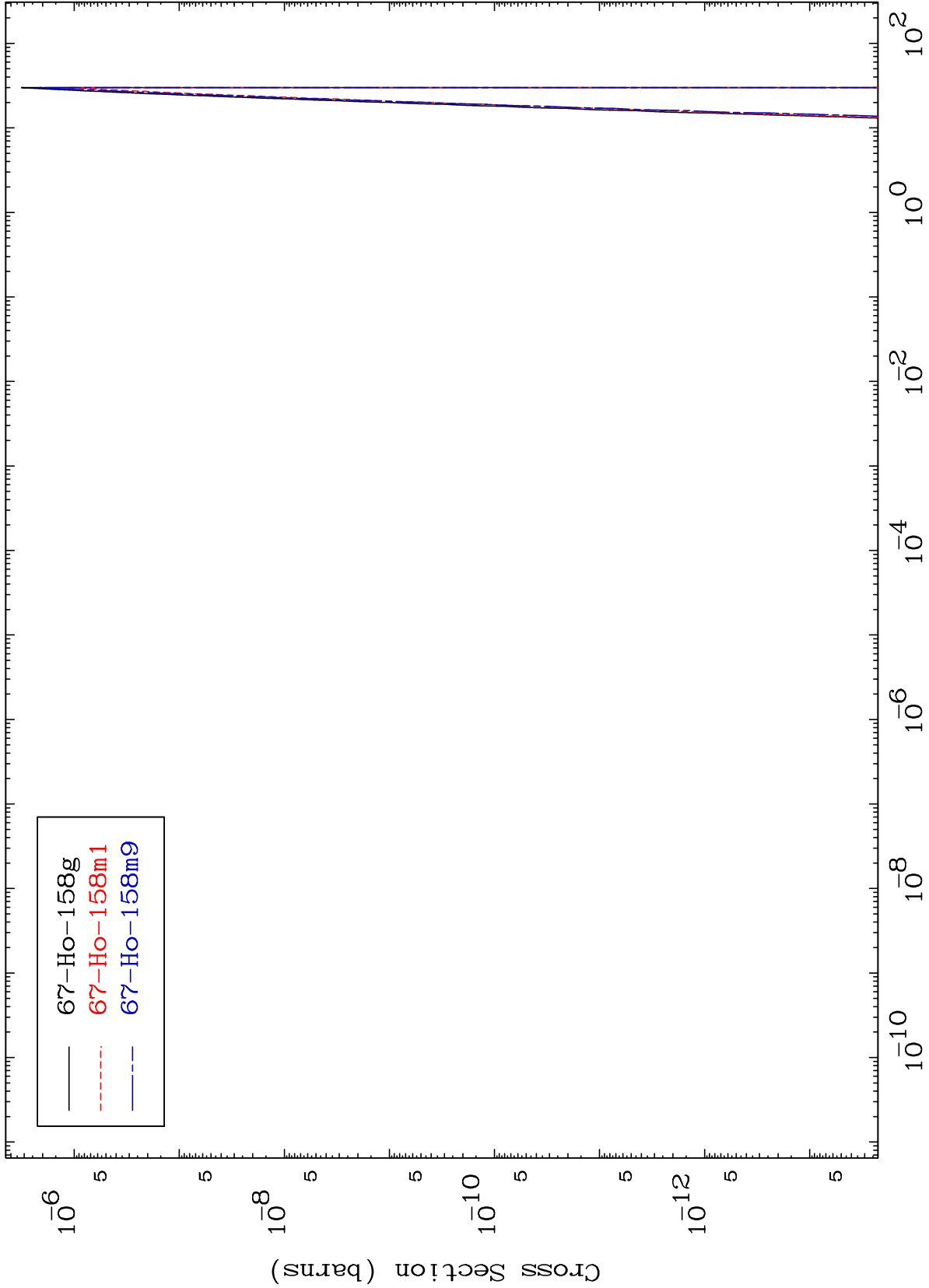


MAT 7100

(n,n') 2α

71-Lu-166n

Radionuclide Production Cross Section

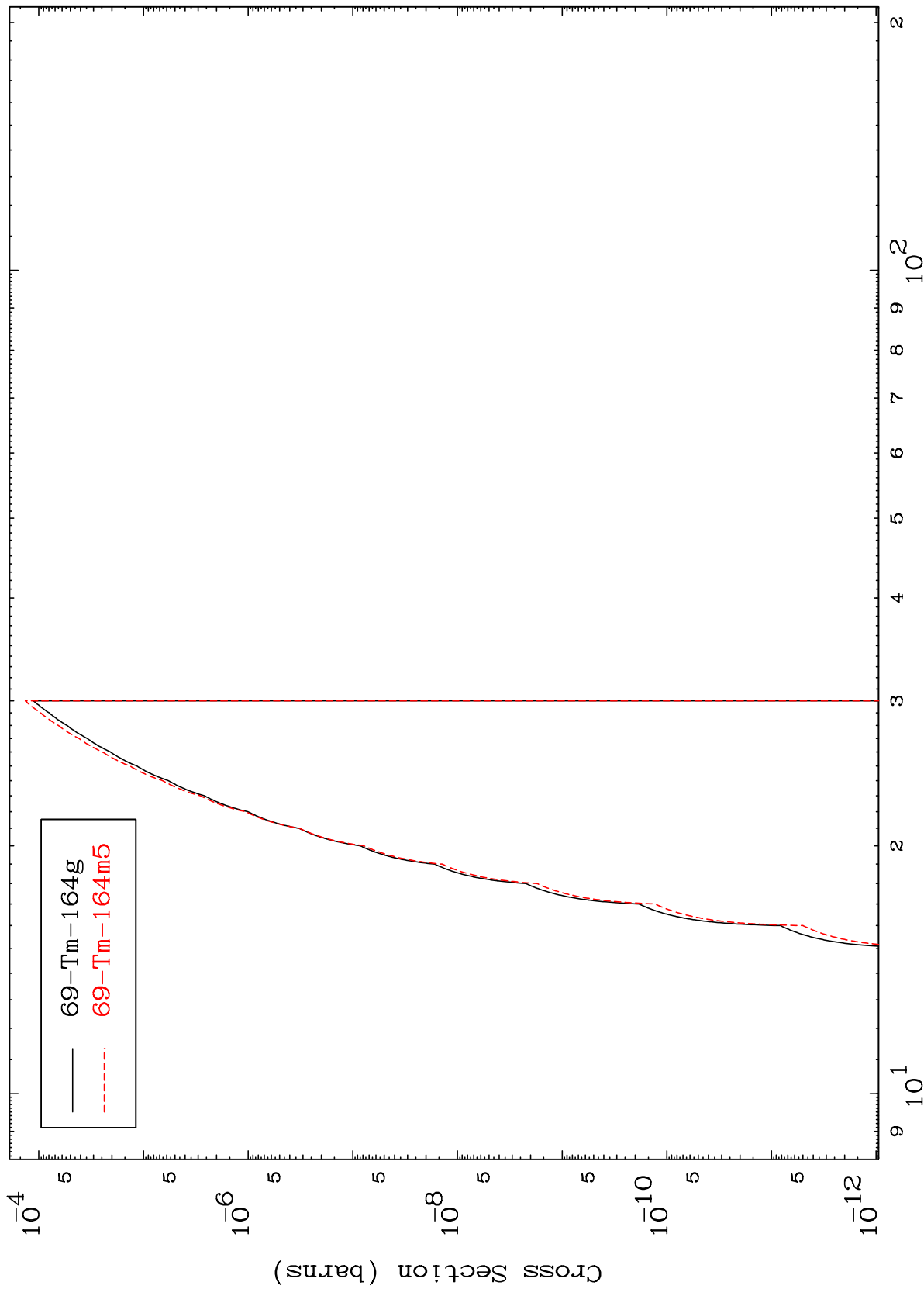


MAT 7100

(n,2n) p

71-Lu-166n

Radionuclide Production Cross Section



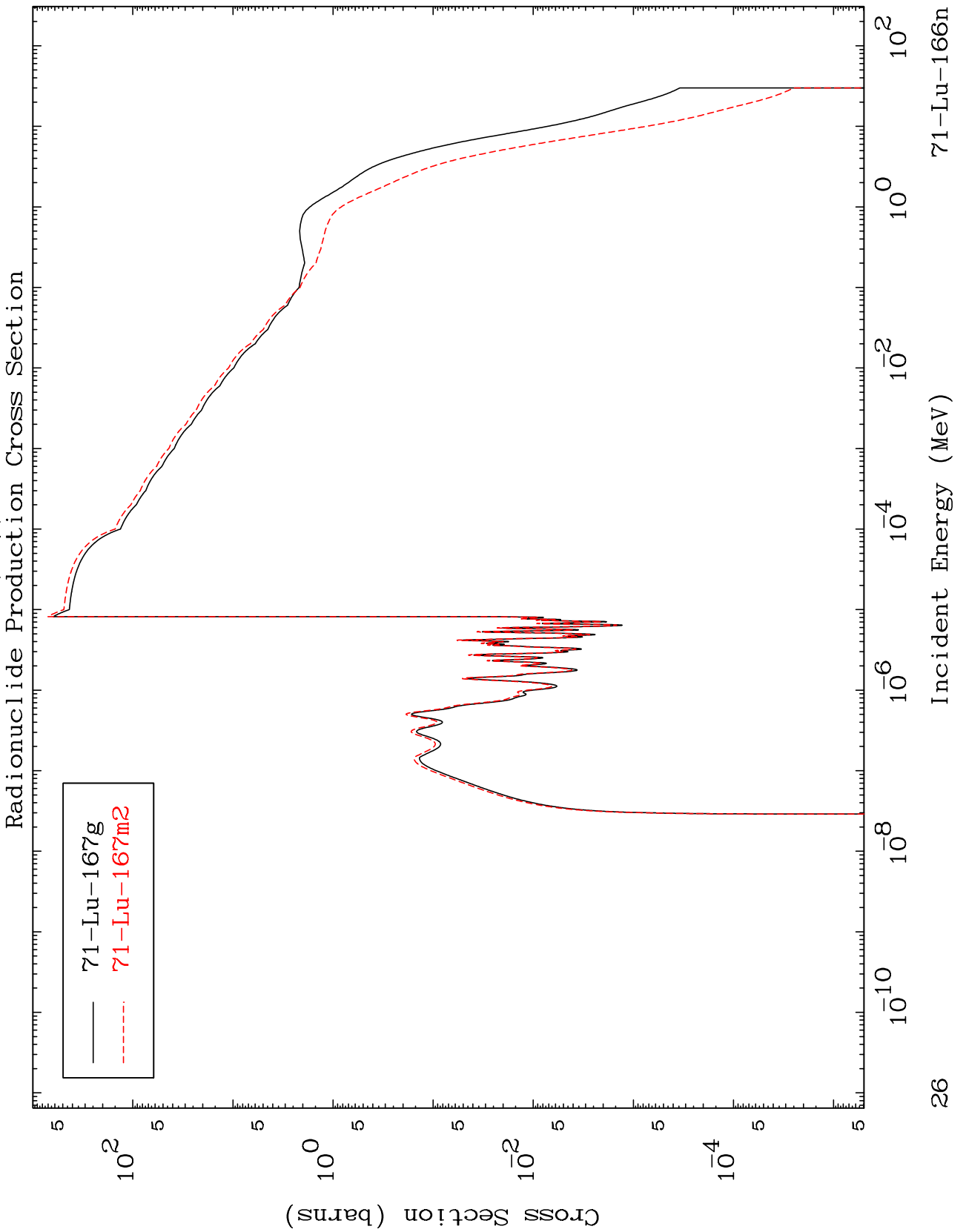
25

Incident Energy (MeV)

71-Lu-166n

MAT 7100

$^{71}\text{Lu-166n}$

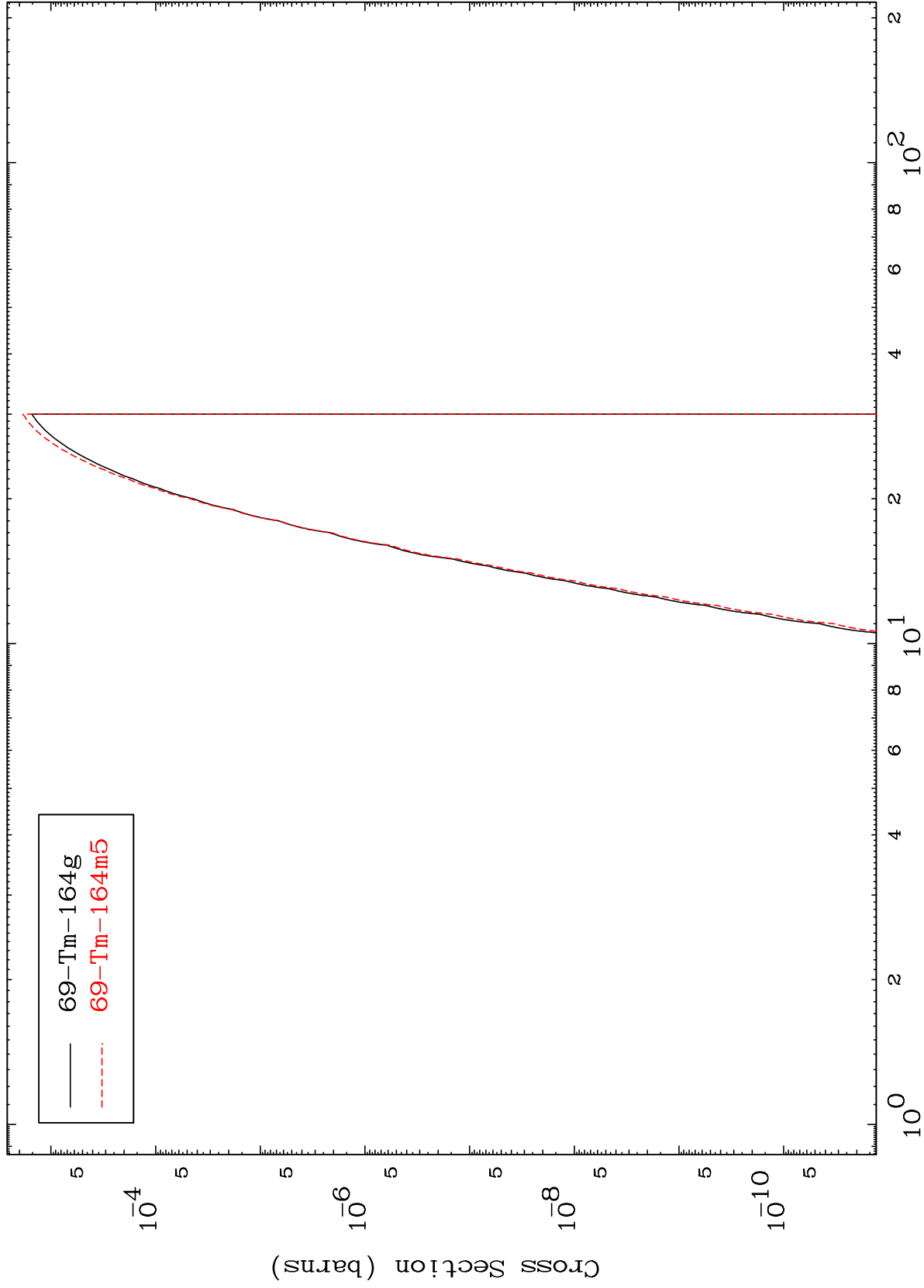


MAT 7100

(n,He-3)

71-Lu-166n

Radionuclide Production Cross Section



27

Incident Energy (MeV)

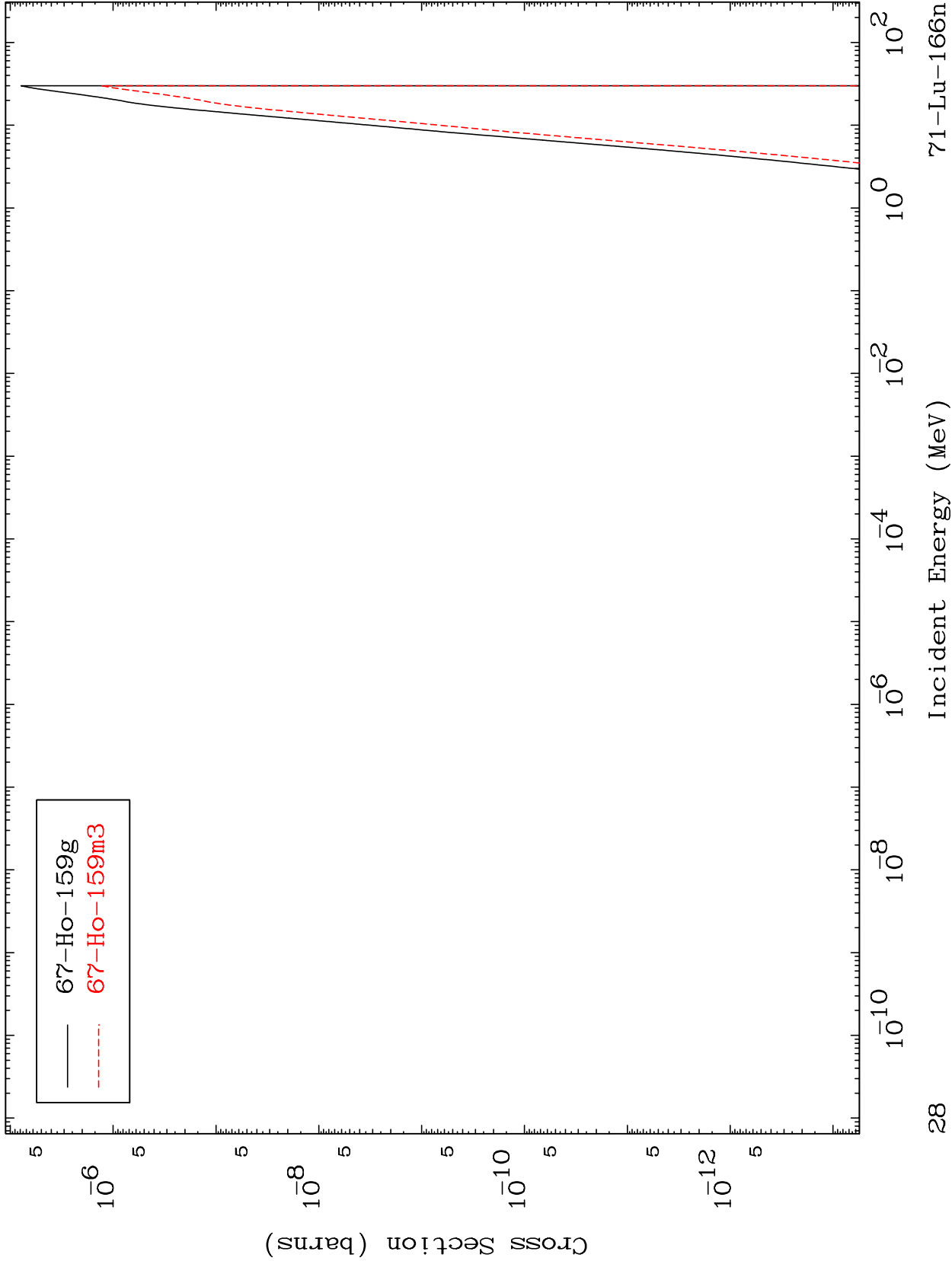
71-Lu-166n

MAT 7100

(n,2α)

<sup>71</sup>Lu-166n

Radionuclide Production Cross Section

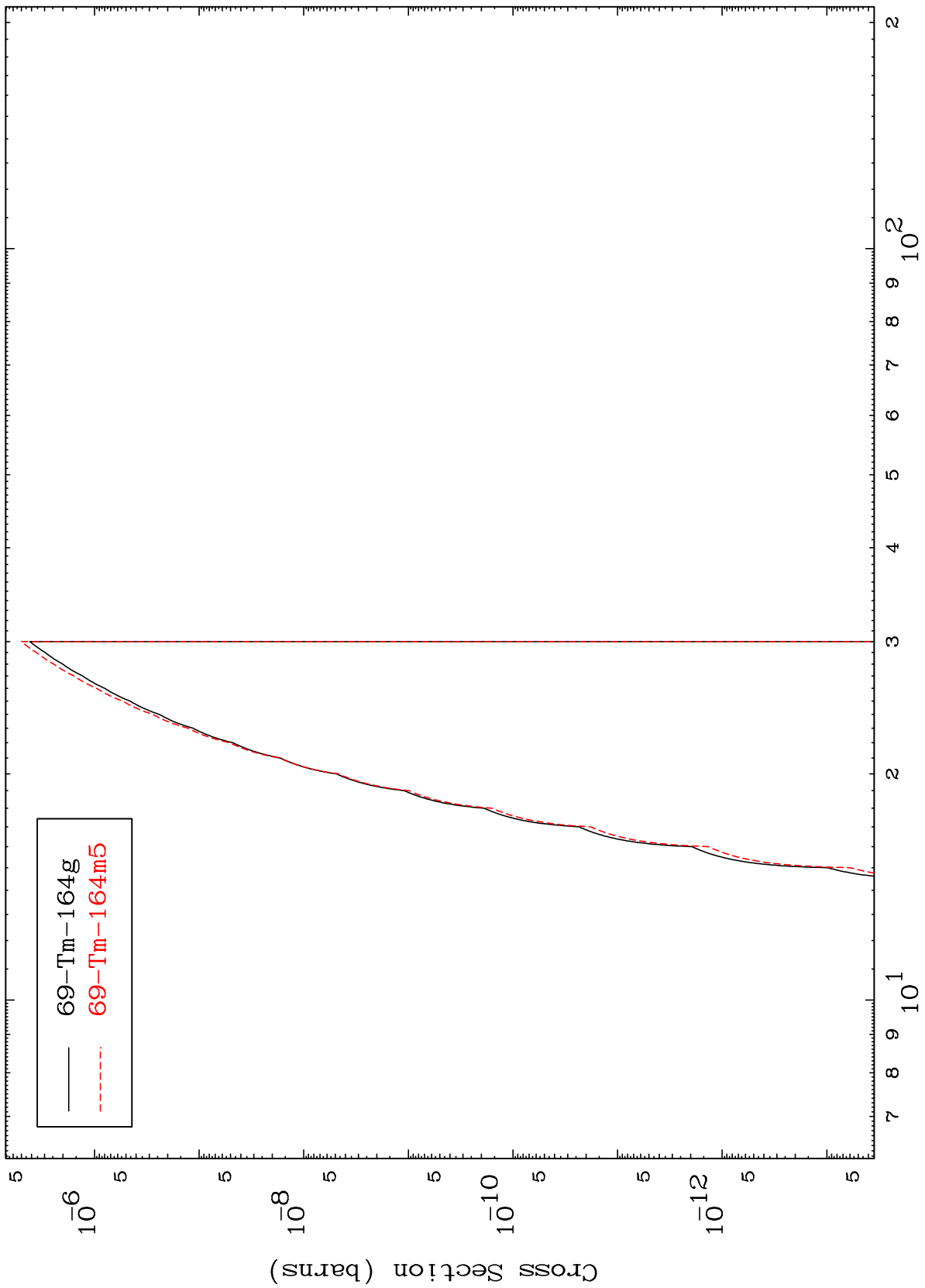


MAT 7100

(n,p) d

<sup>71</sup>Lu-166n

Radionuclide Production Cross Section



29

Incident Energy (MeV)

<sup>71</sup>Lu-166n