

Program EVALPLOT
(Version 2021-1)

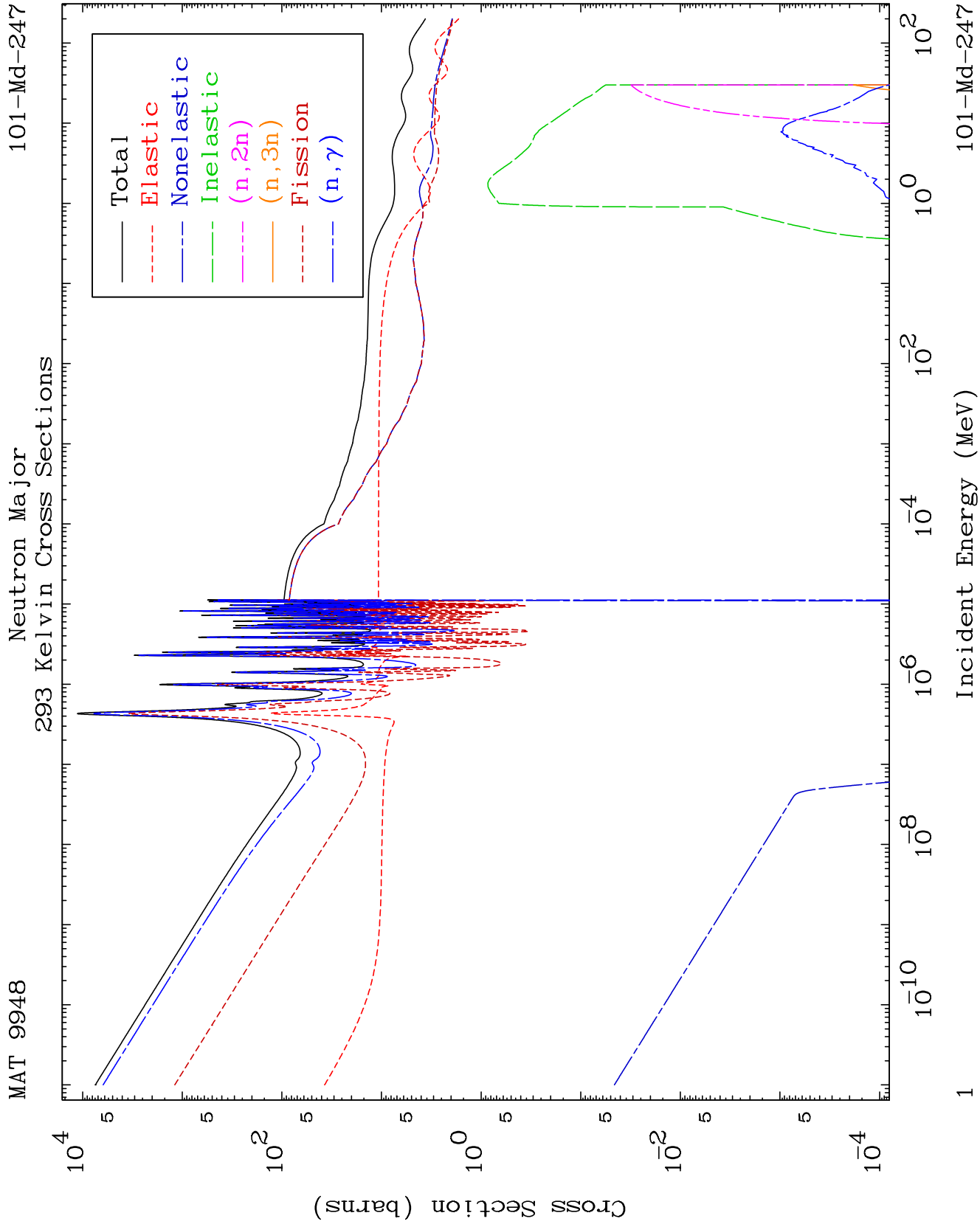
by

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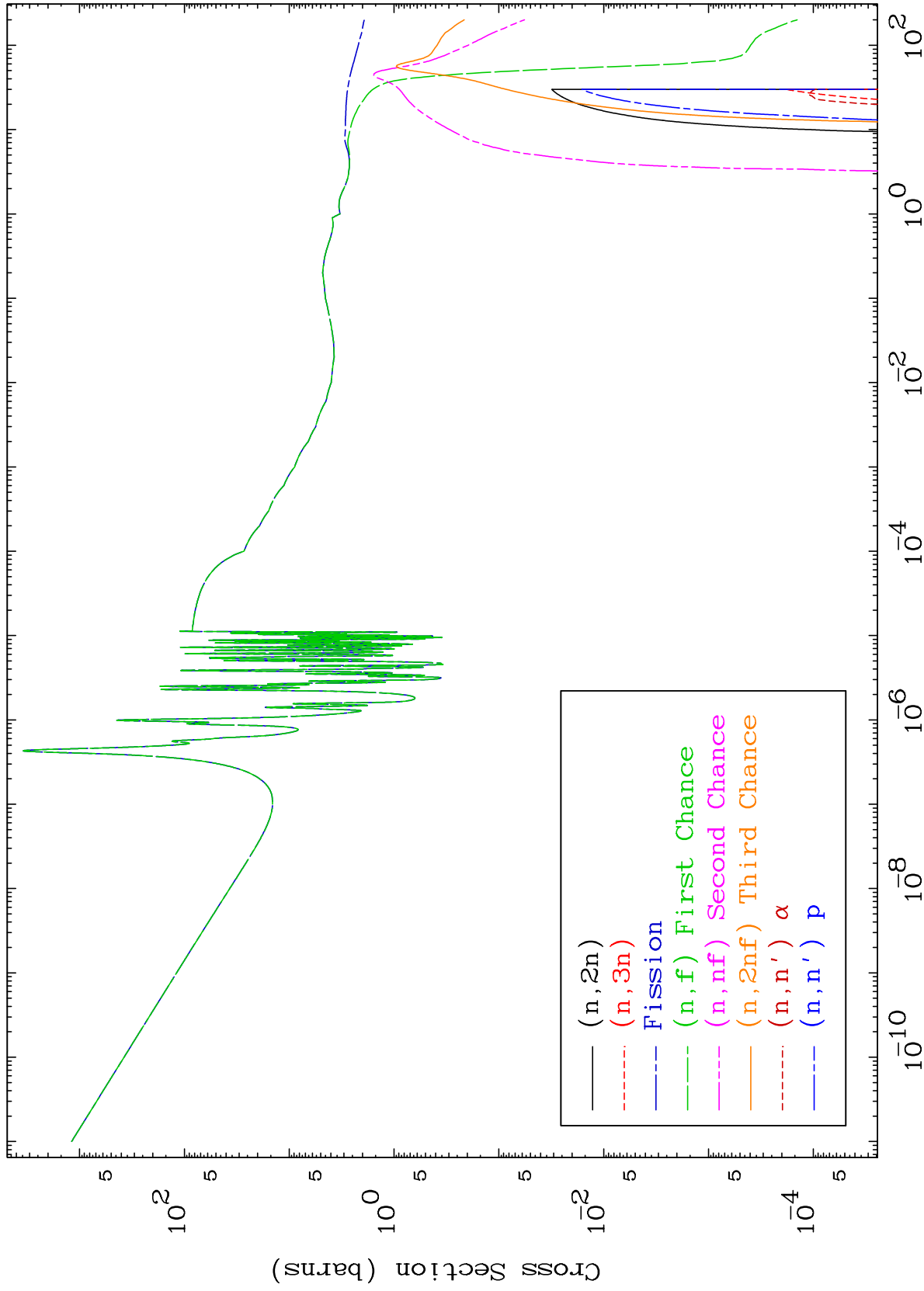
Press Mouse Button to Start

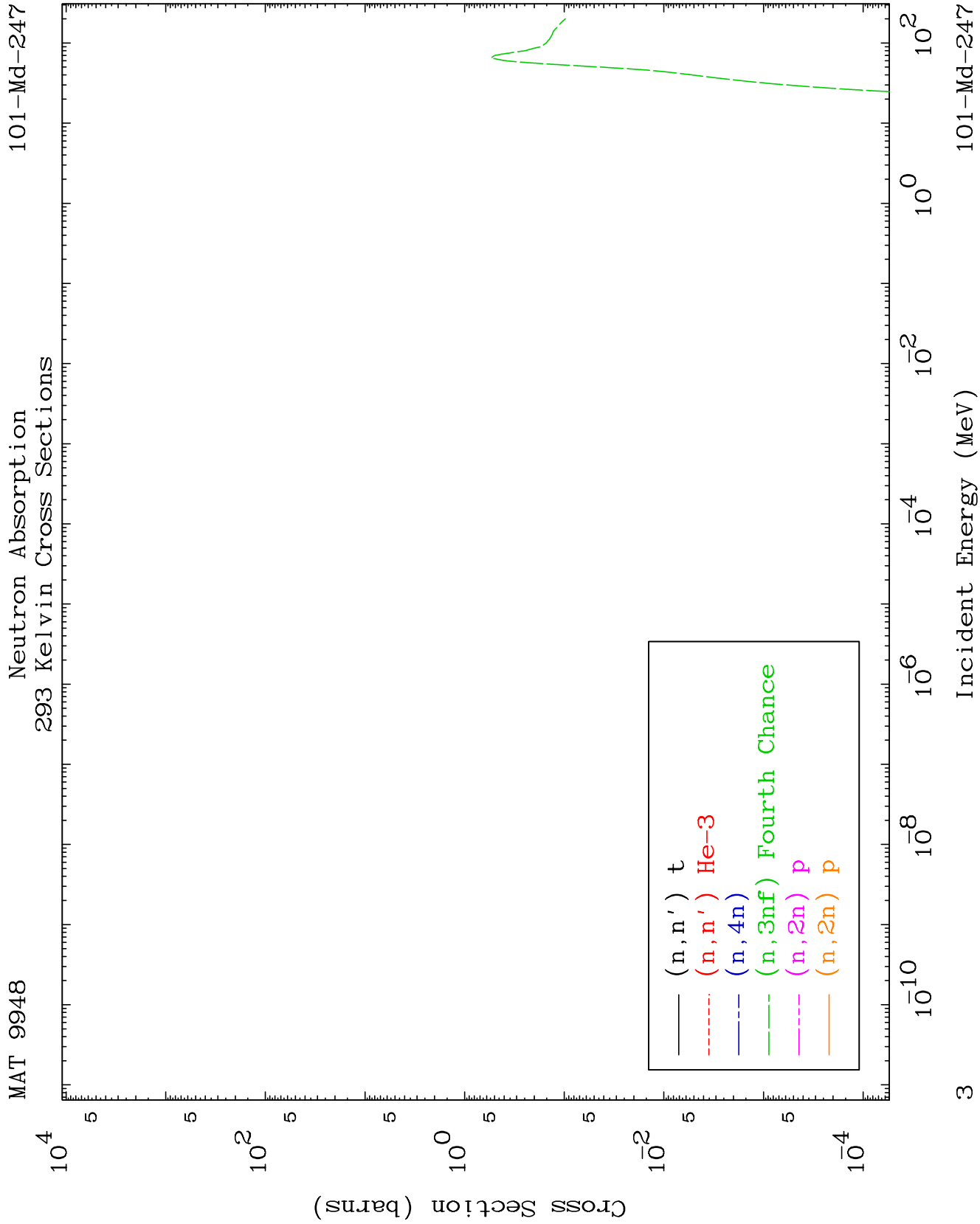


MAT 9948

Neutron Absorption
293 Kelvin Cross Sections

101-Md-247

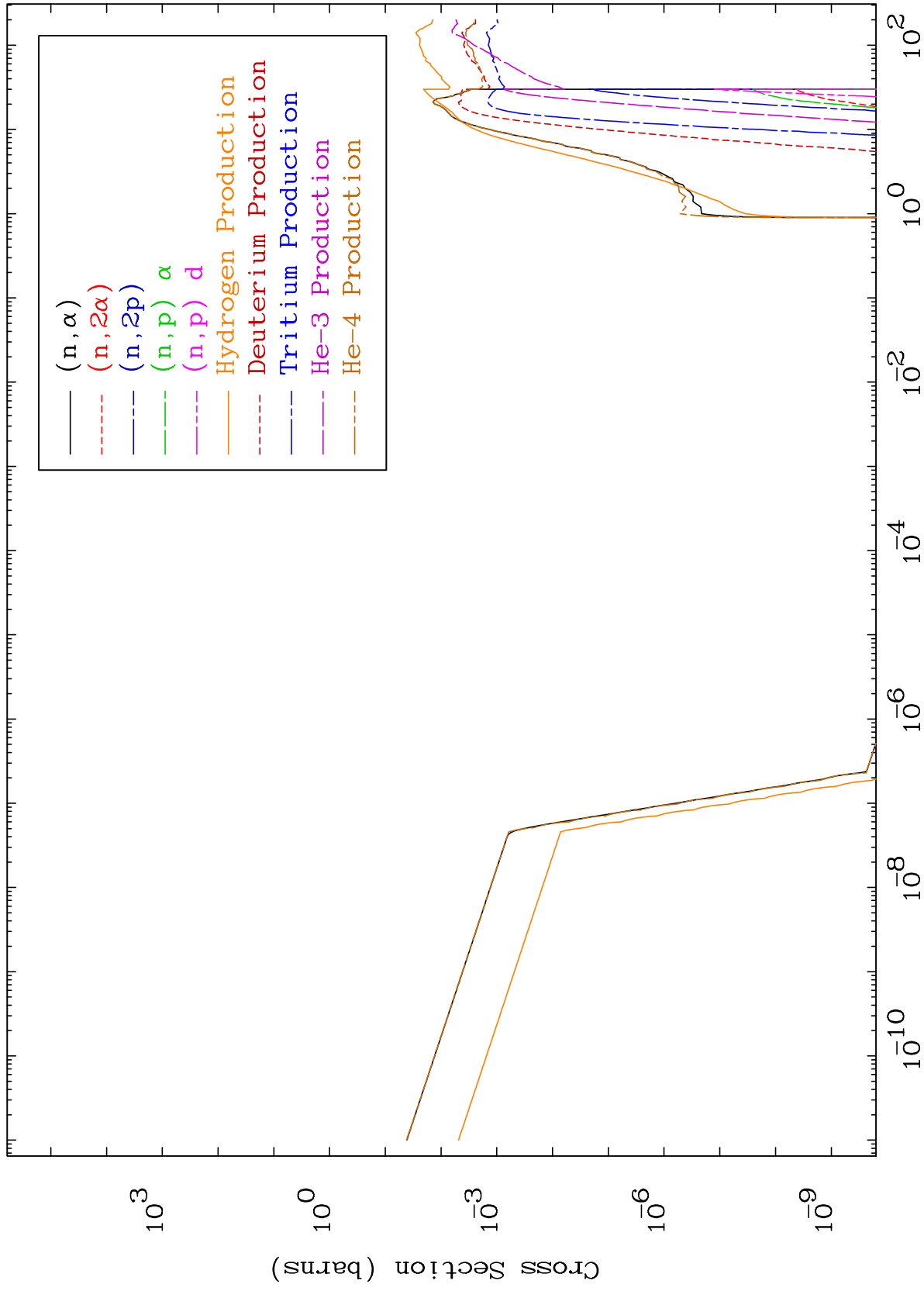


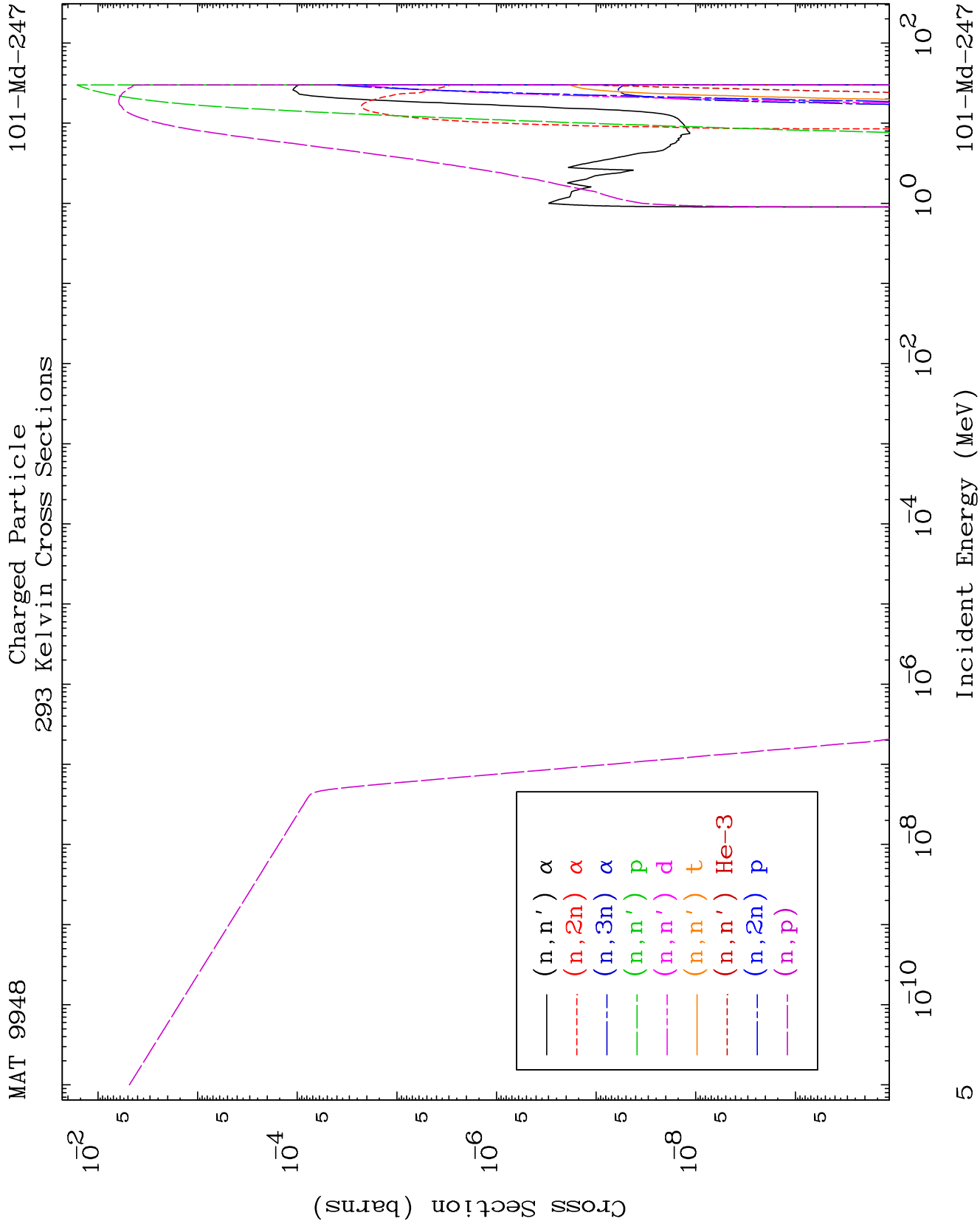


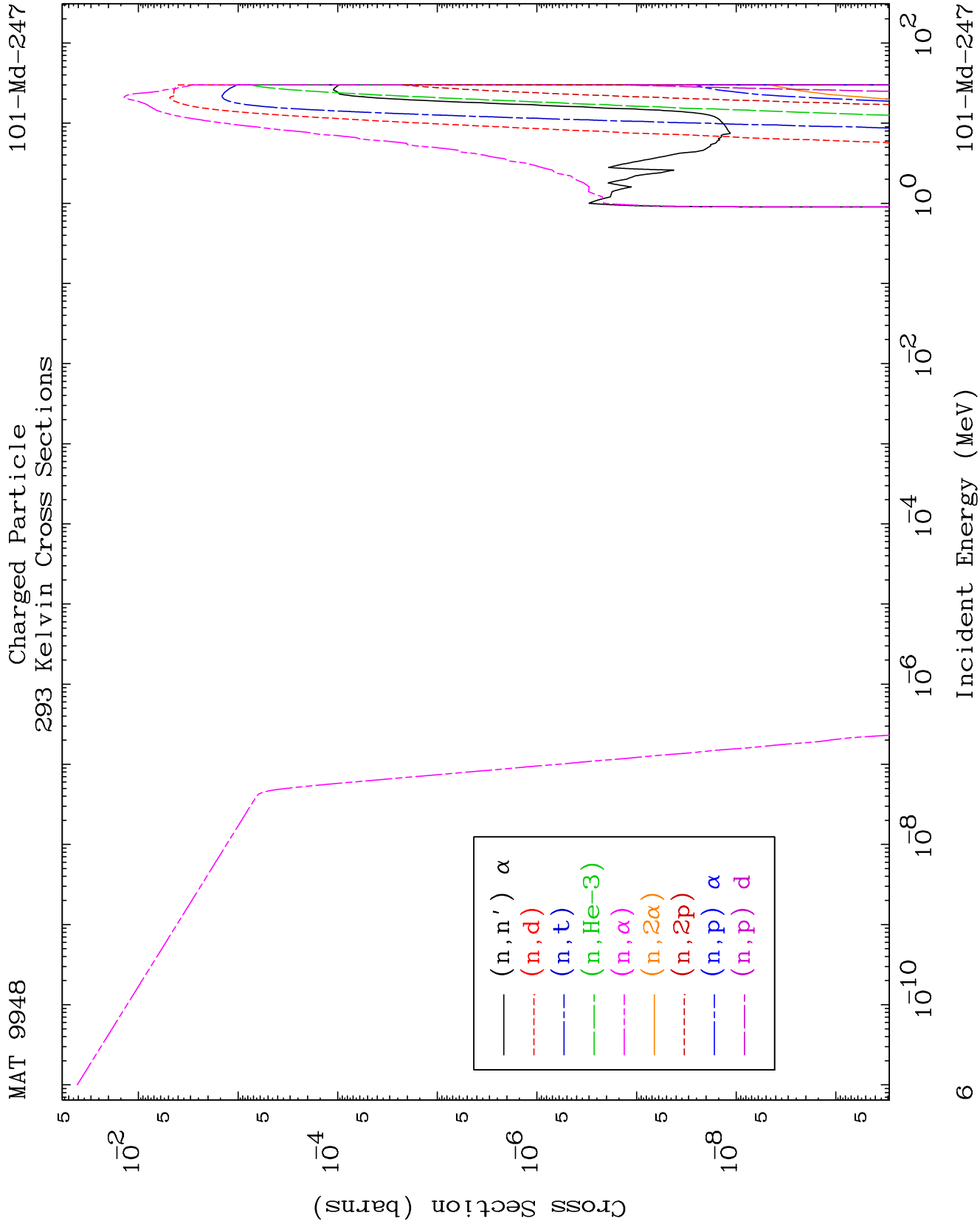
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Neutron Absorption
293 Kelvin Cross Sections

101-Md-247



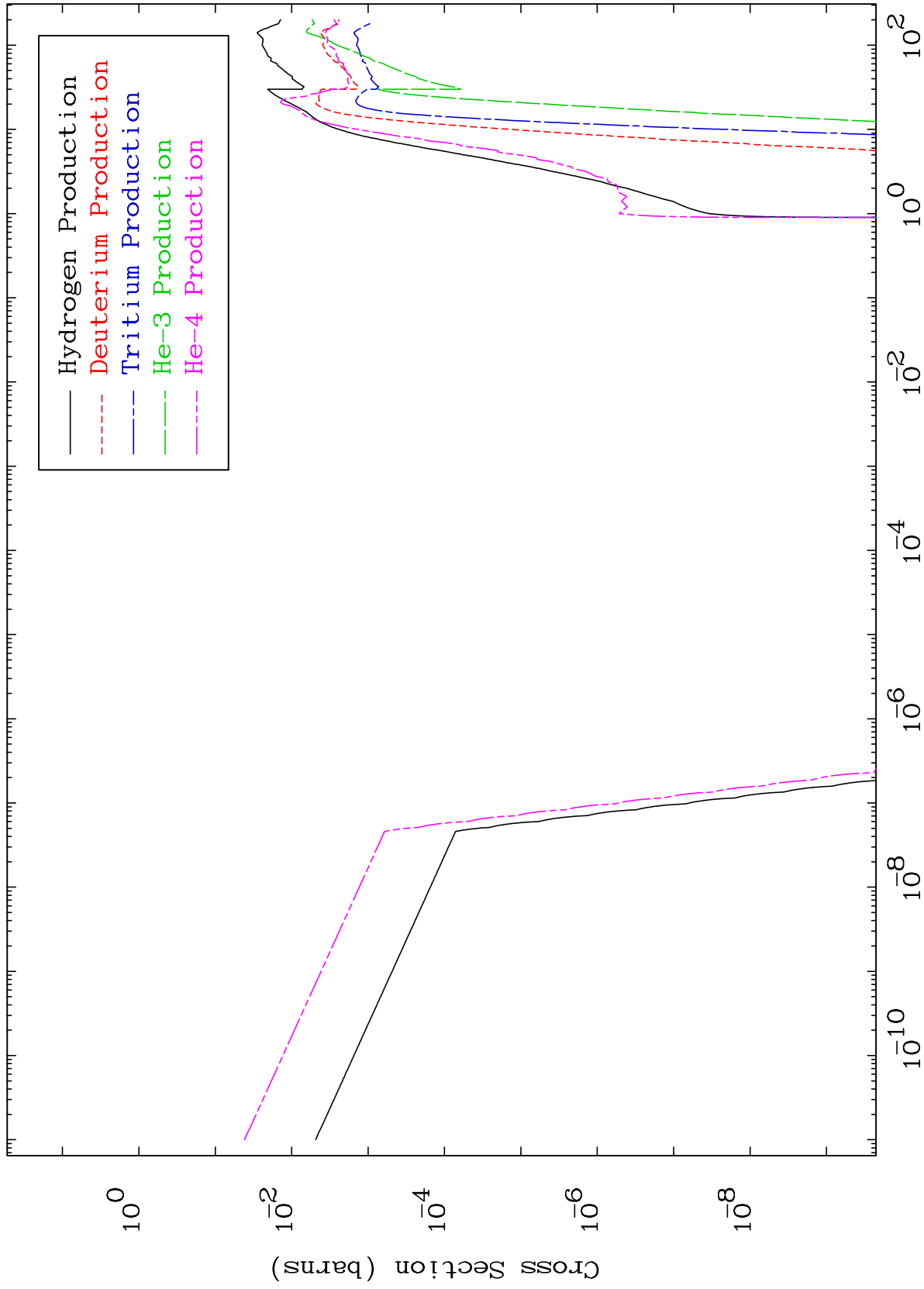




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Particle Production
293 Kelvin Cross Sections

101-Md-247



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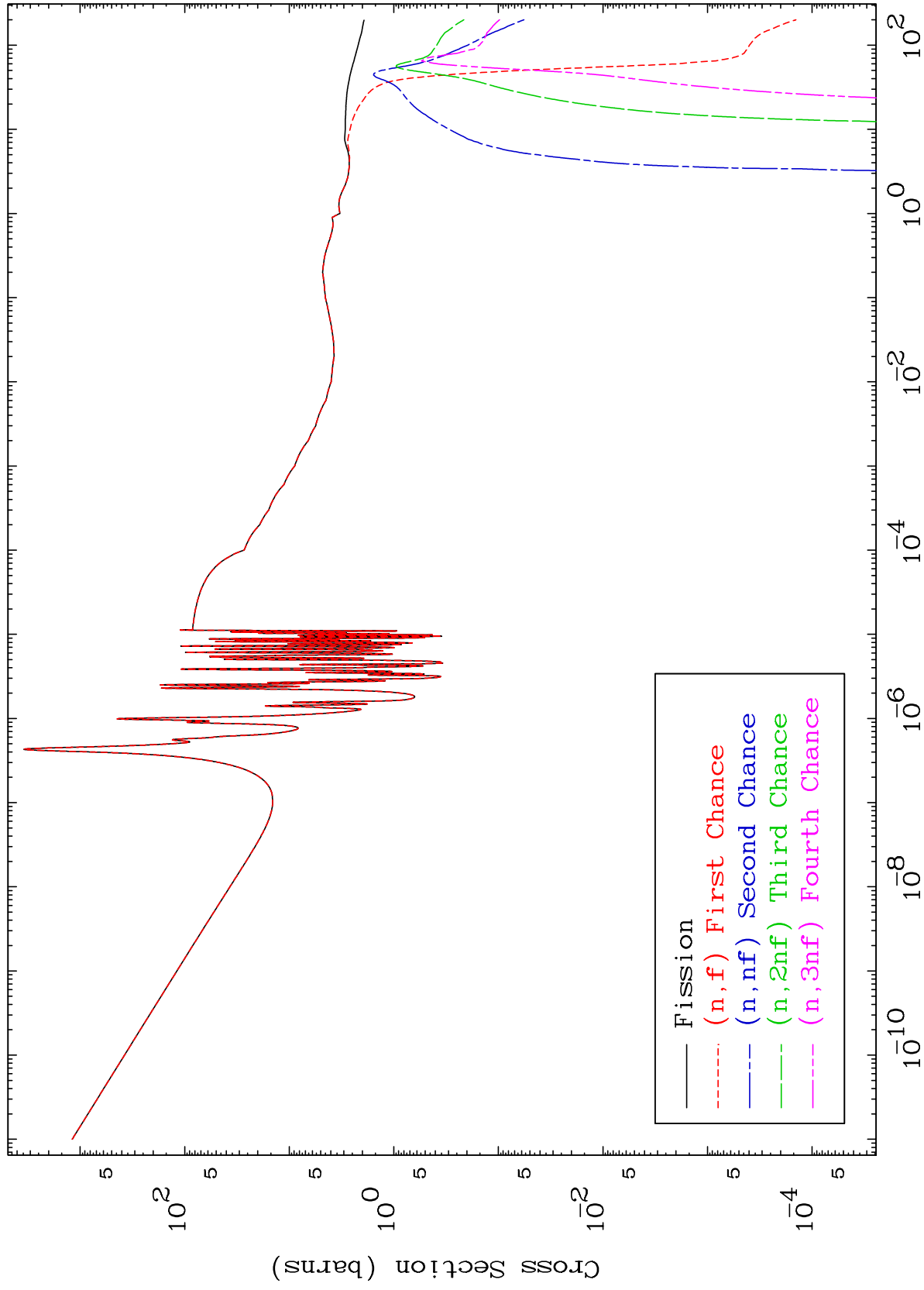
Incident Energy (MeV)

101-Md-247

MAT 9948

Fission
293 Kelvin Cross Sections

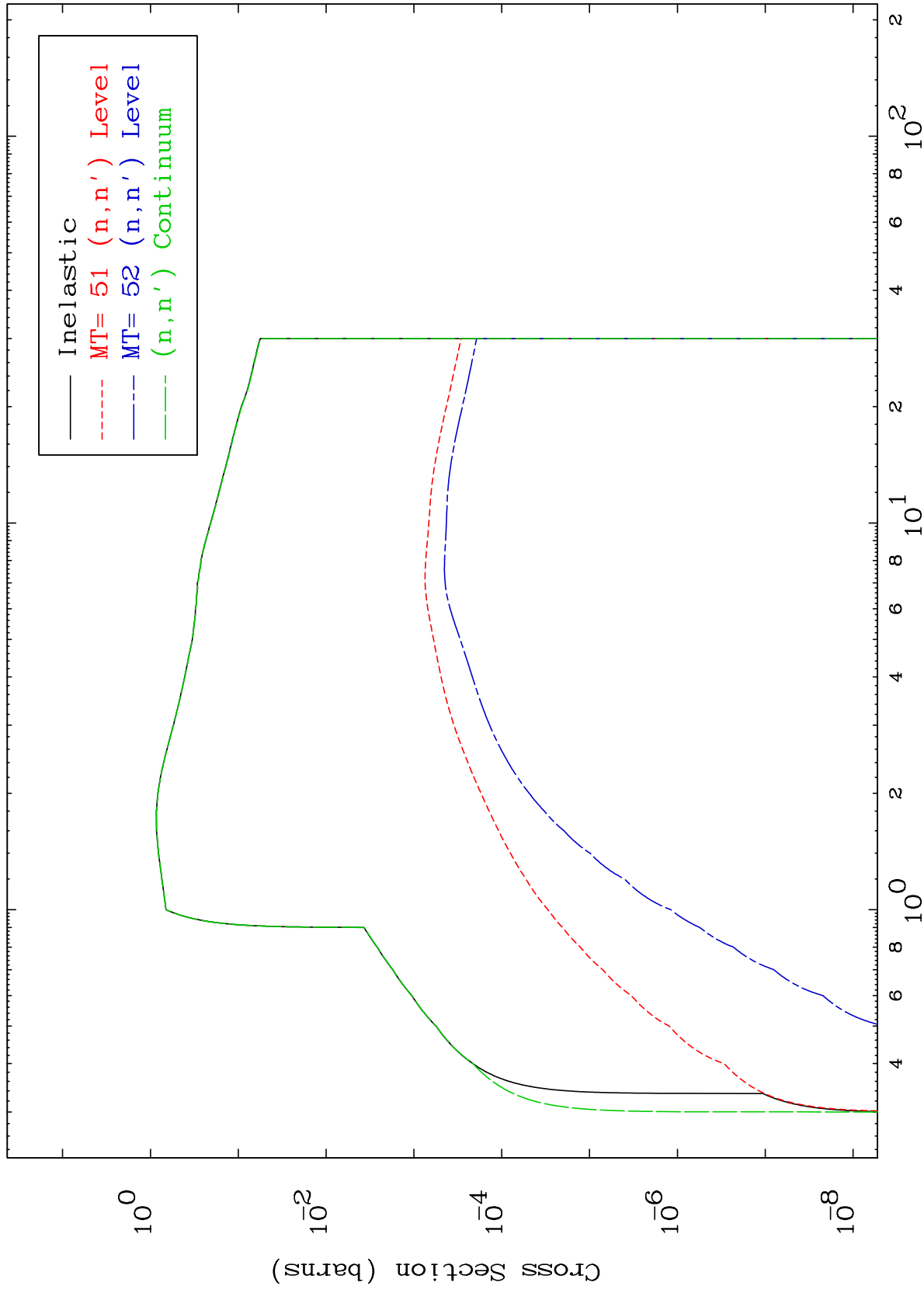
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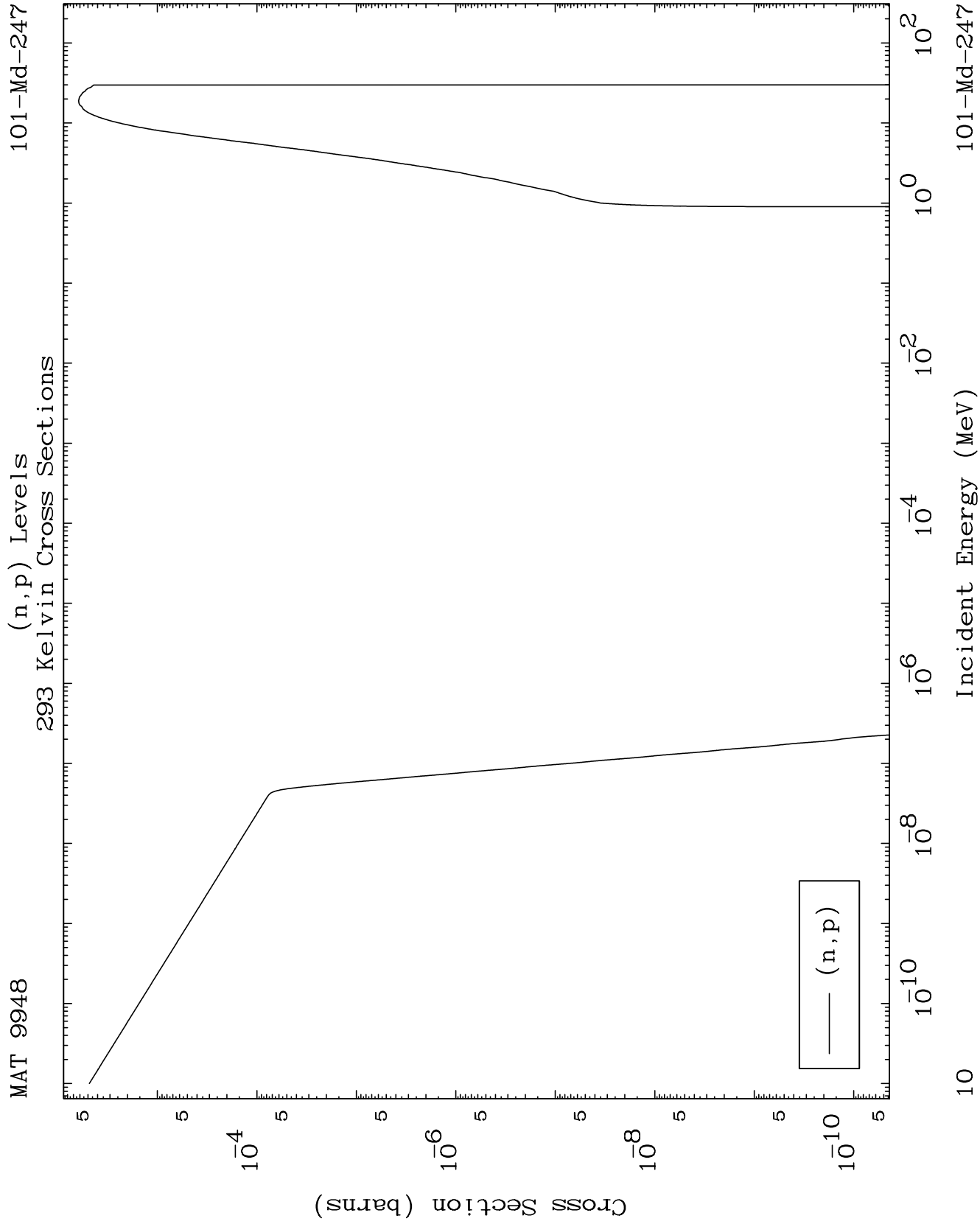


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(n,n') Levels
293 Kelvin Cross Sections

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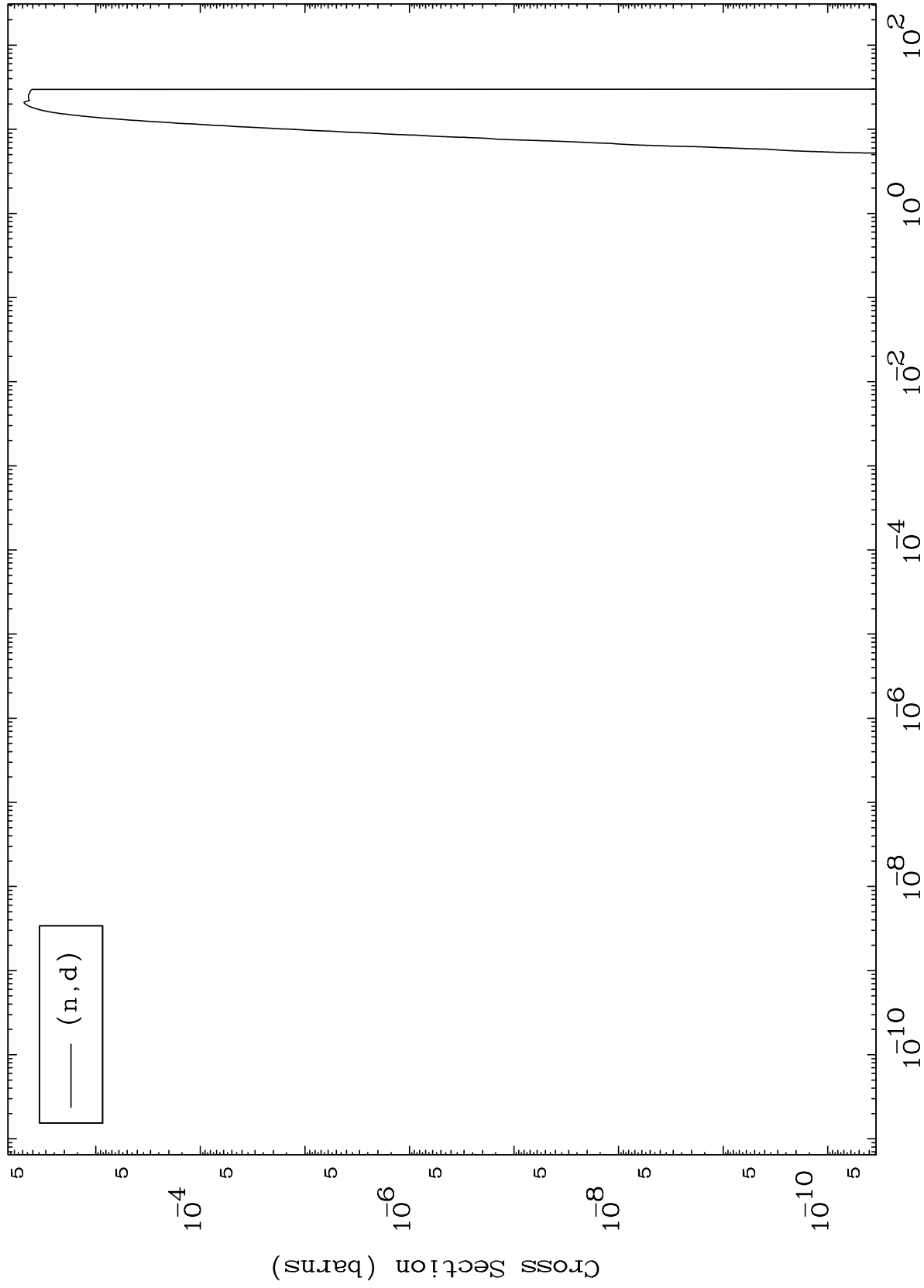




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(n,d) Levels
293 Kelvin Cross Sections

101-Md-247



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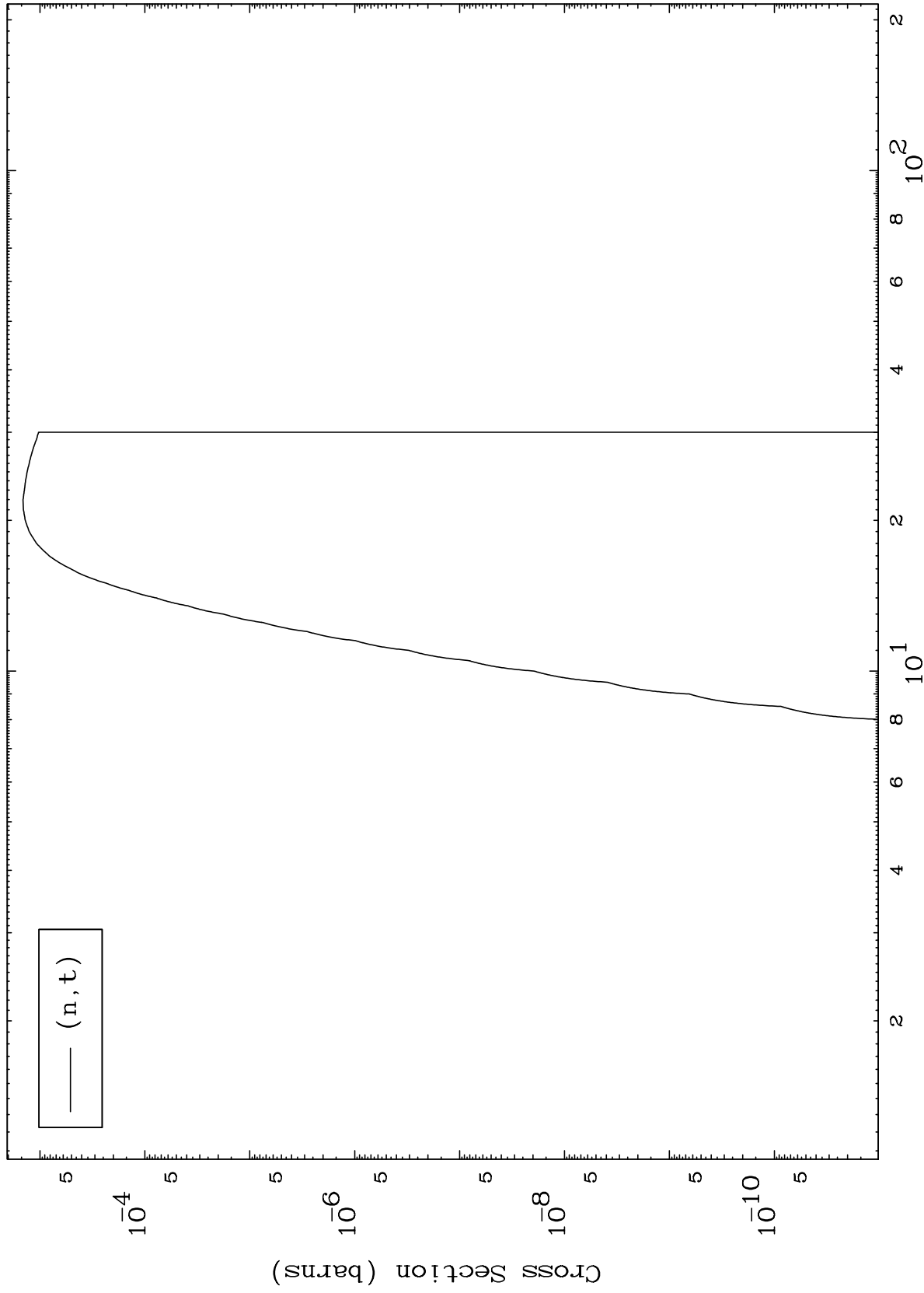
Incident Energy (MeV)

101-Md-247

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(n,t) Levels
293 Kelvin Cross Sections

101-Md-247



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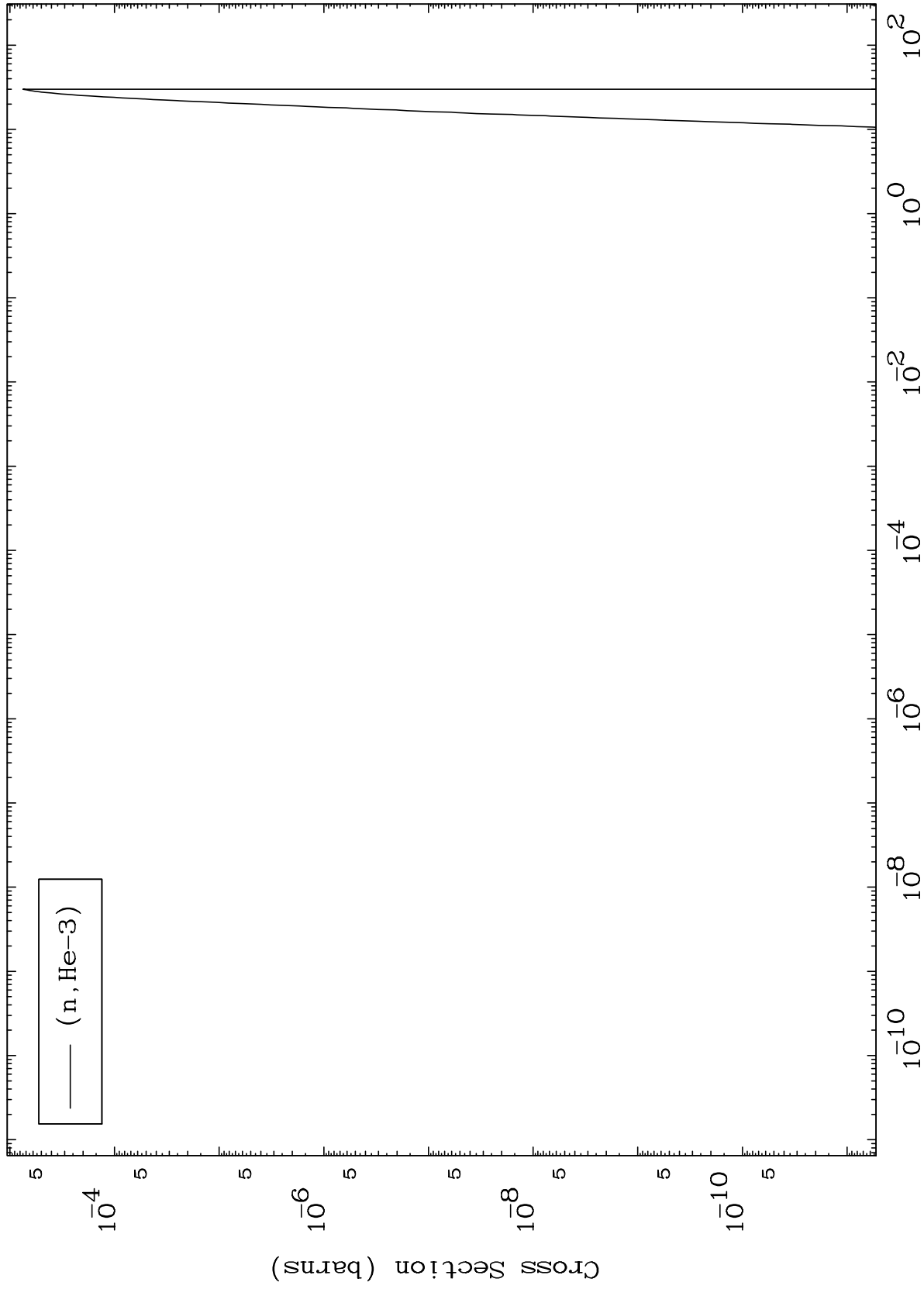
Incident Energy (MeV)

101-Md-247

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(n,He3) Levels
293 Kelvin Cross Sections

101-Md-247



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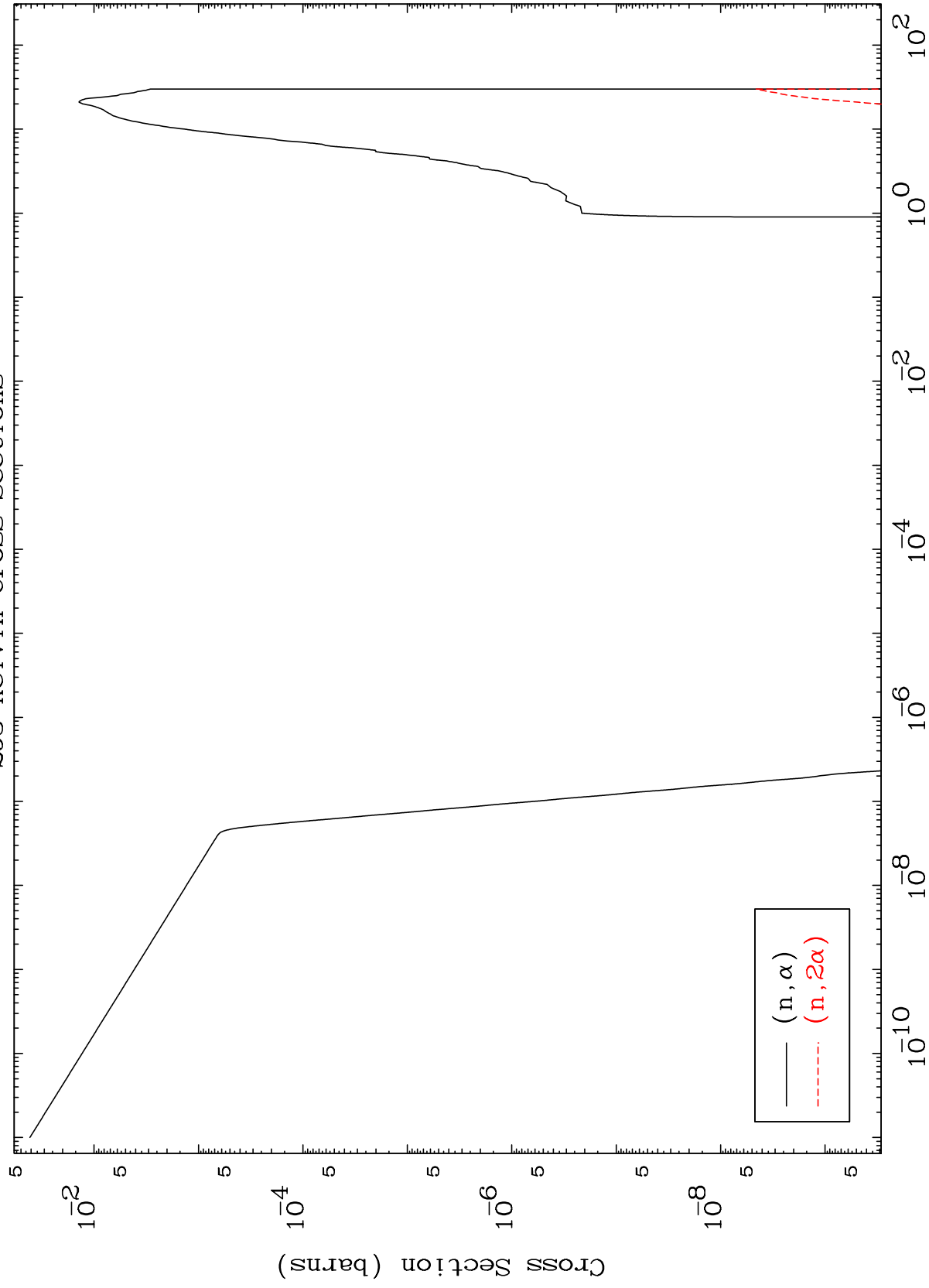
Incident Energy (MeV)

101-Md-247

MAT 9948

(n, α) Levels
293 Kelvin Cross Sections

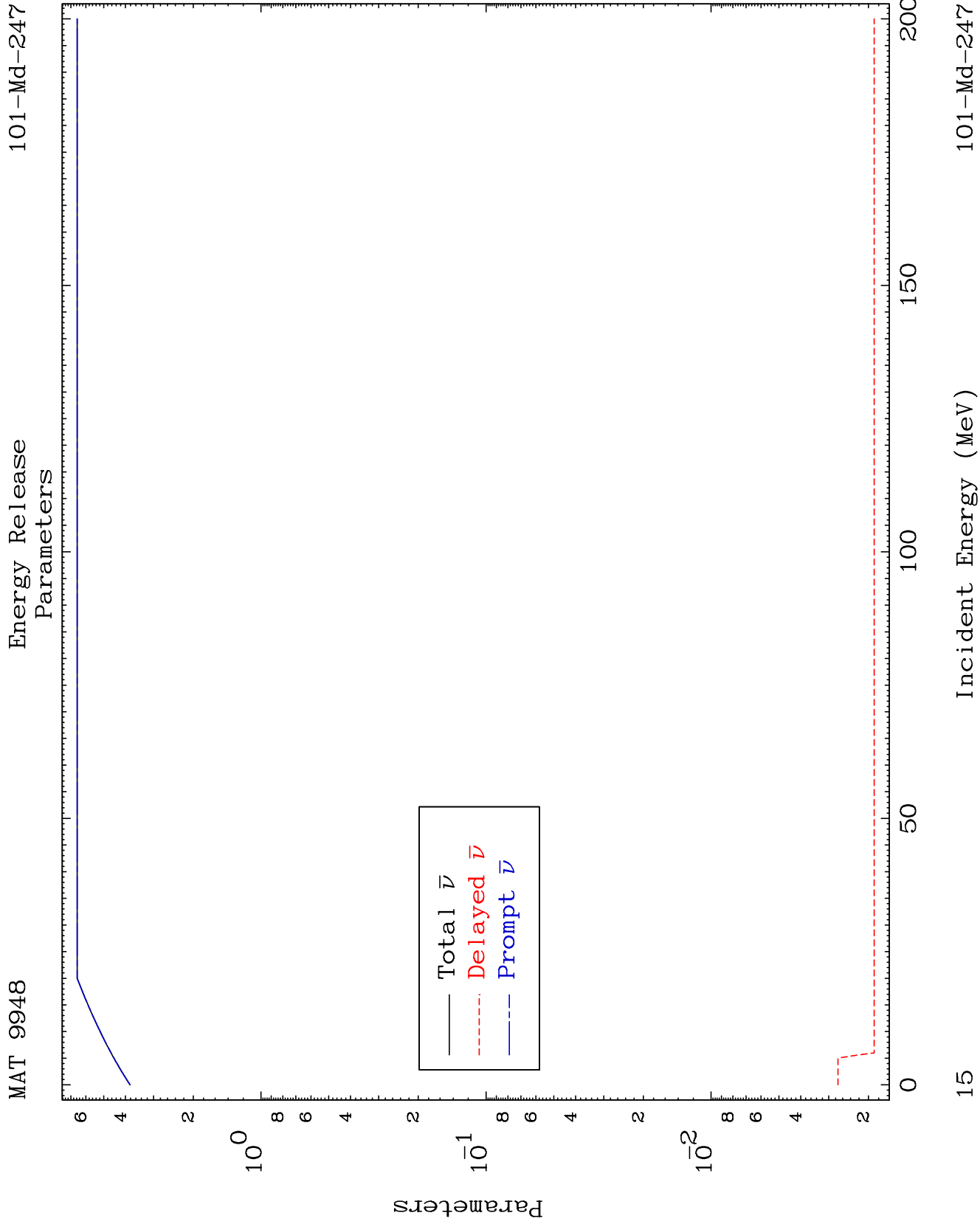
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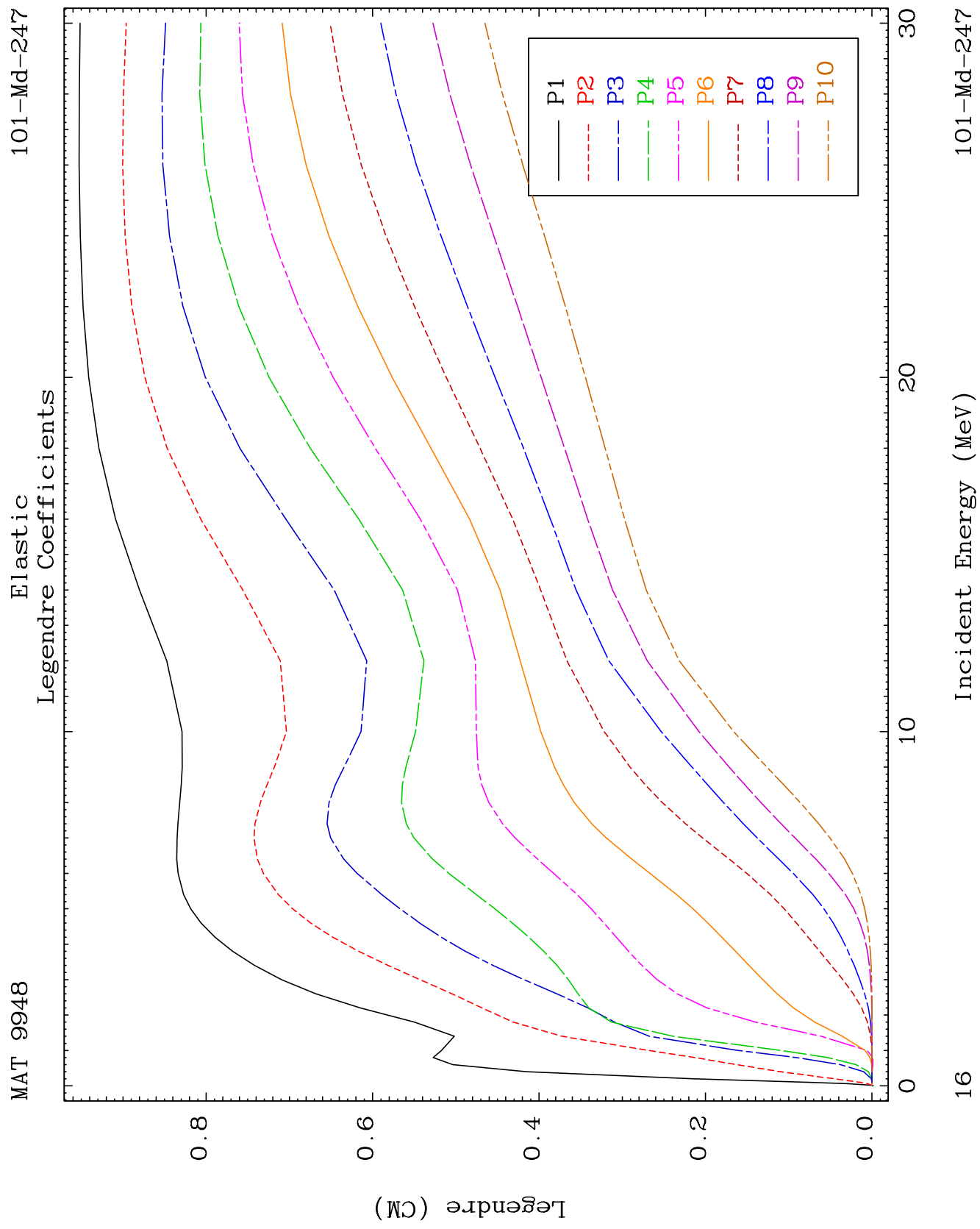


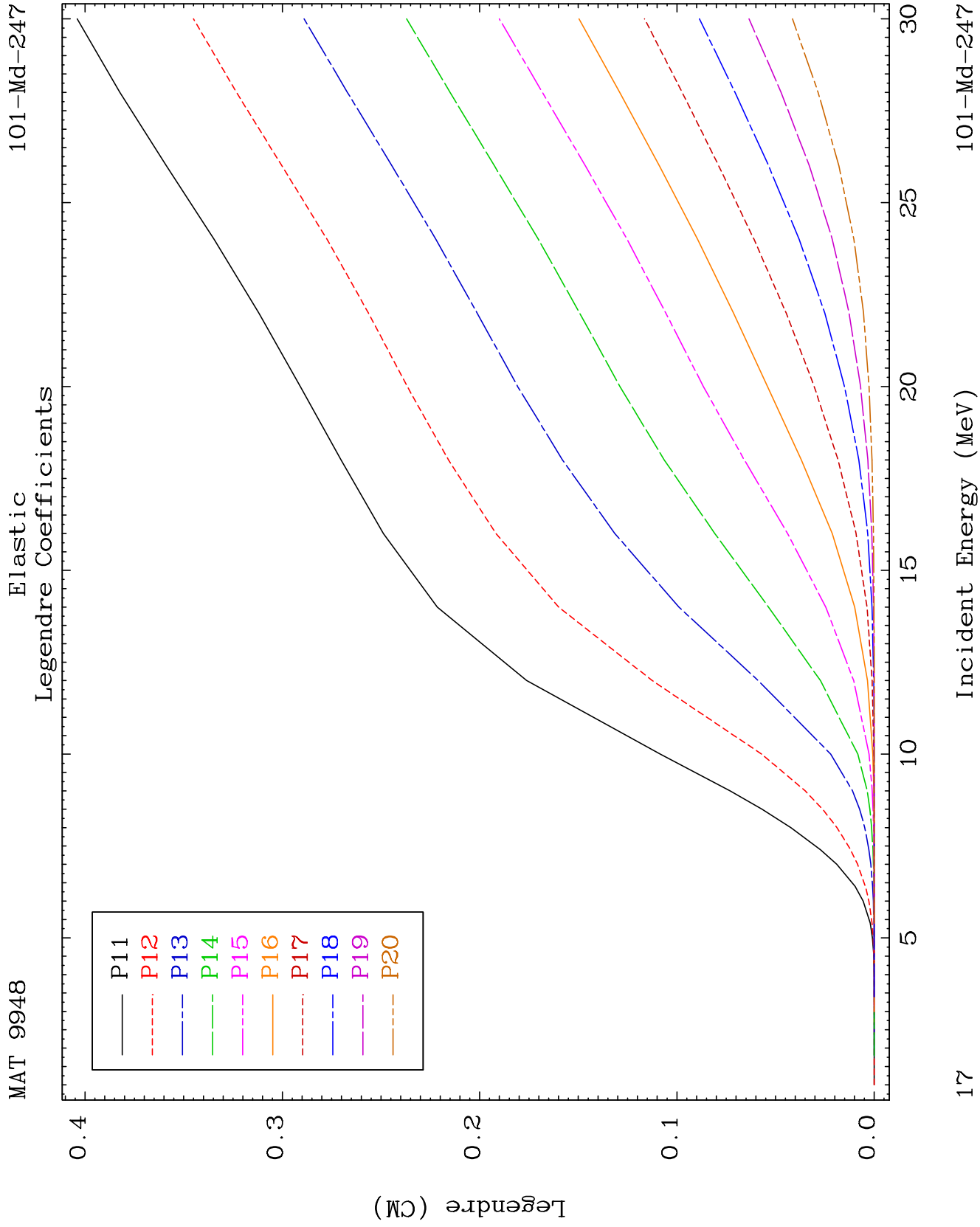
14

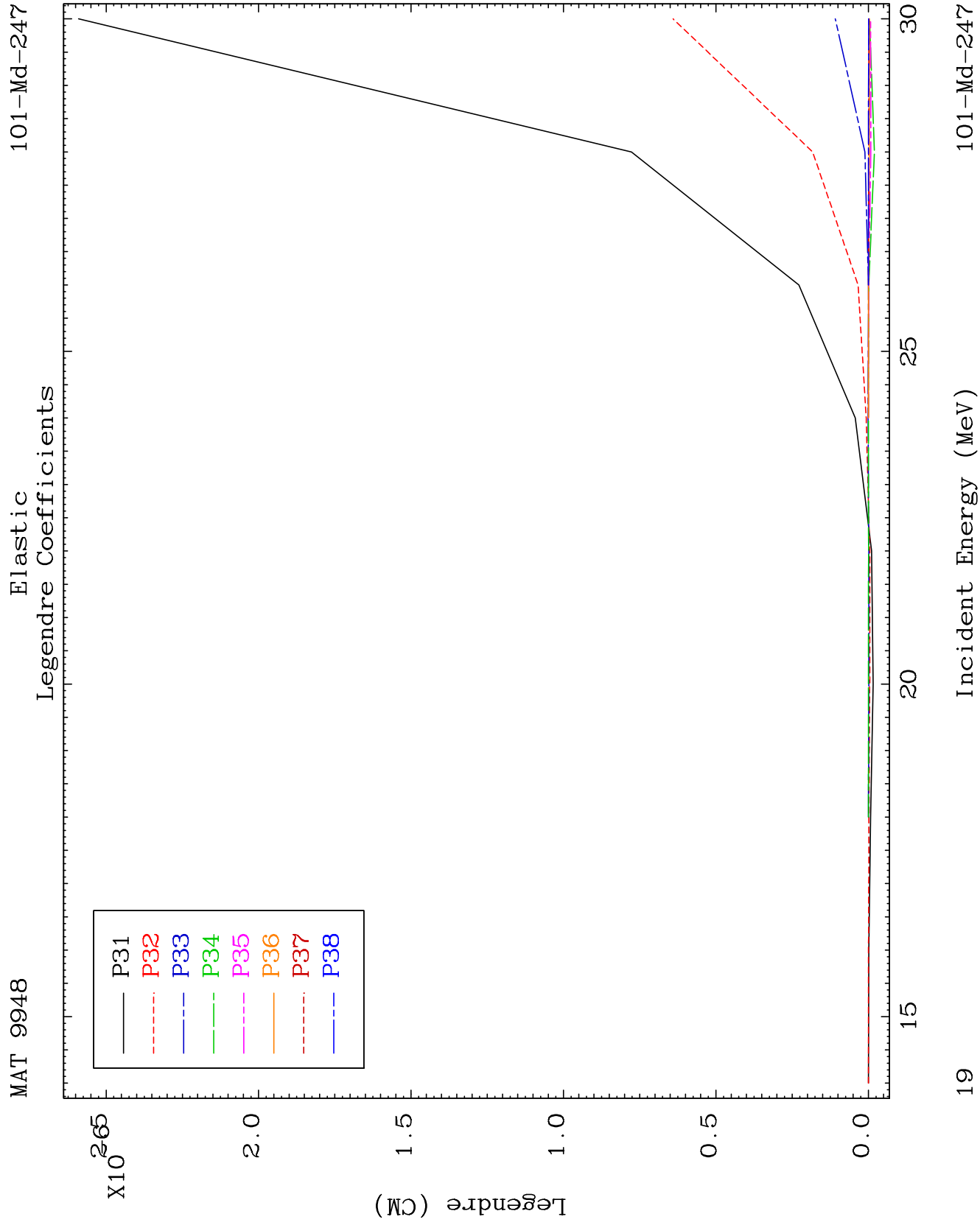
Incident Energy (MeV)

101-Md-247





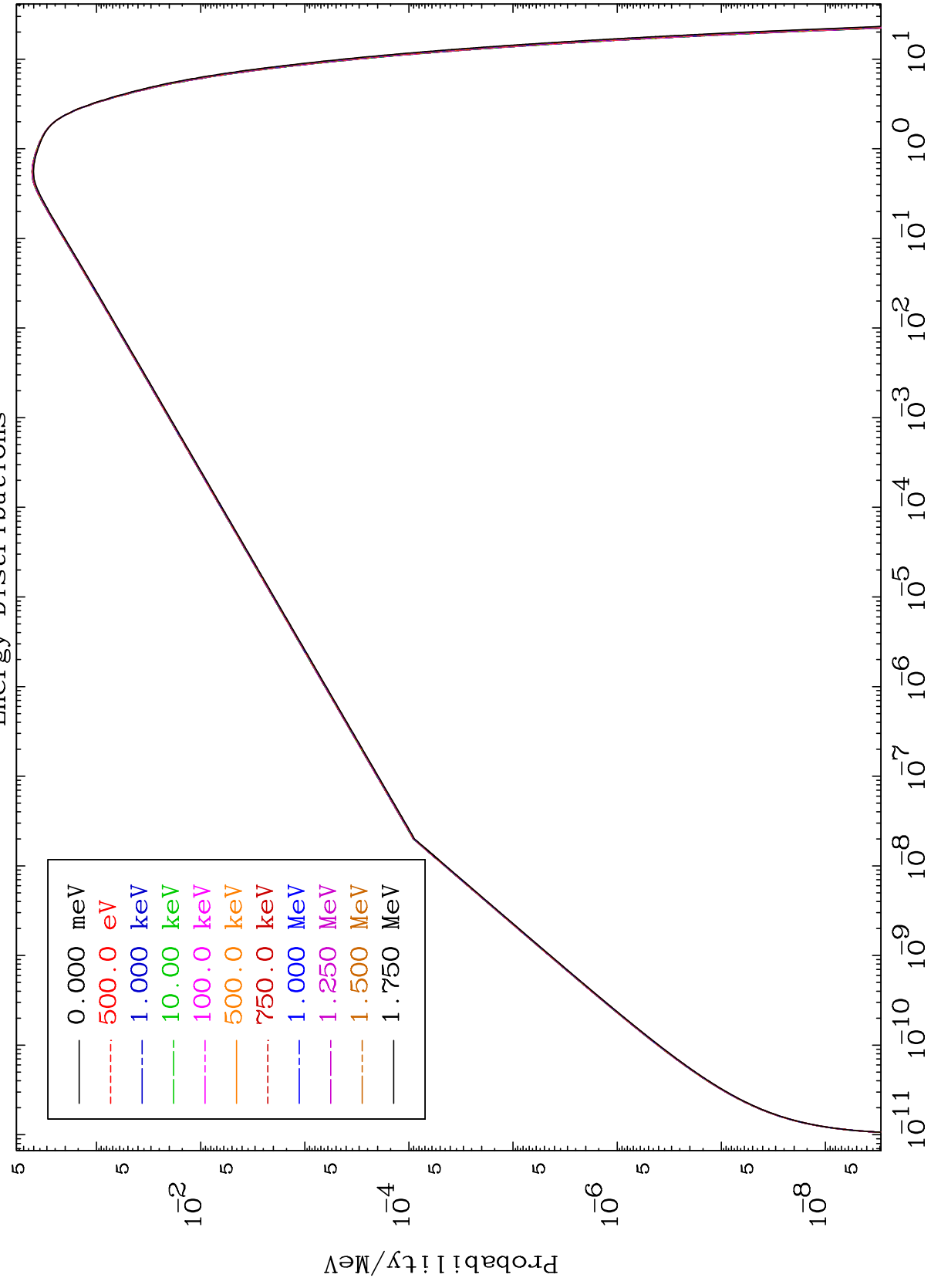




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Fission
Energy Distributions

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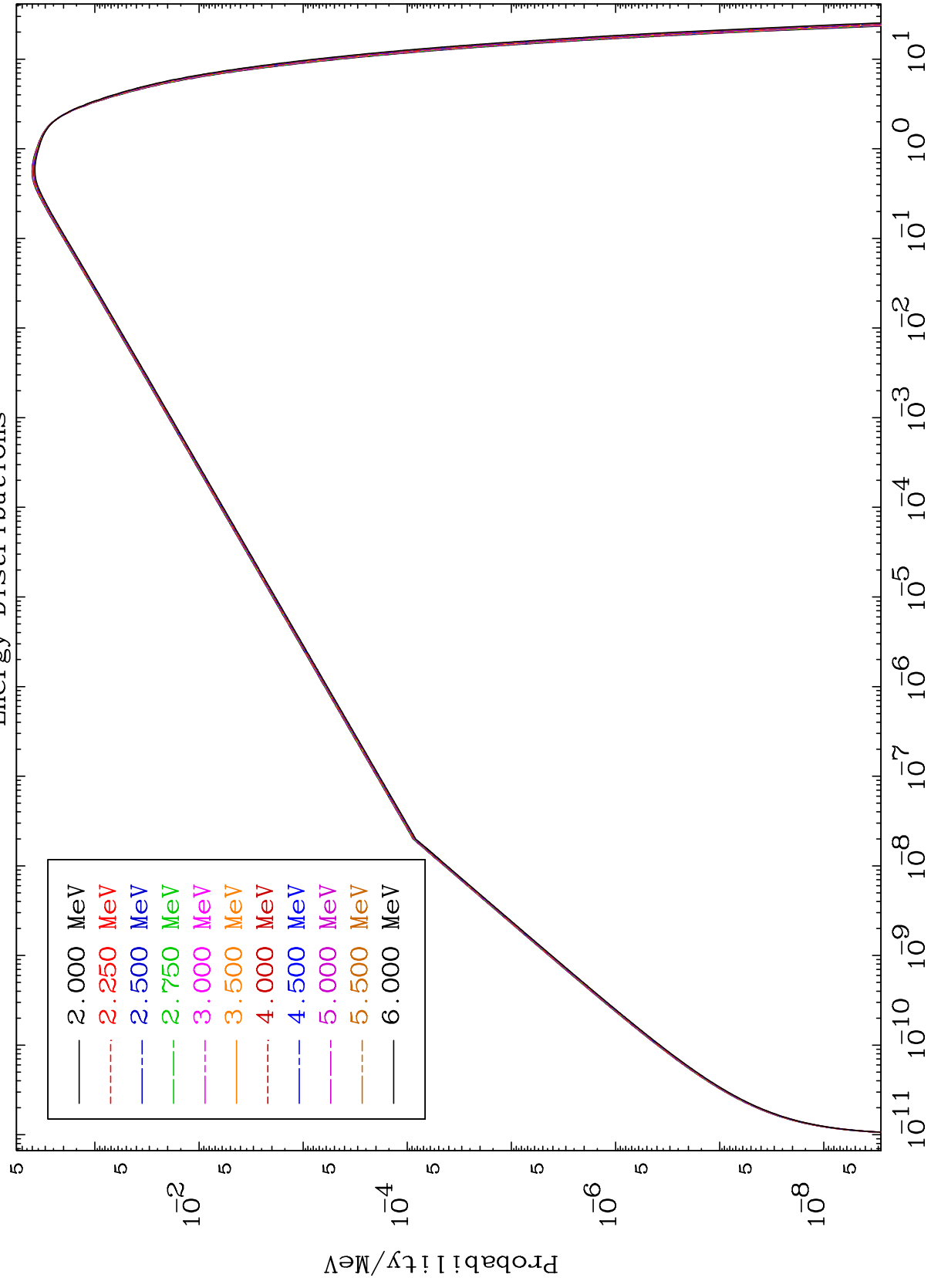


- 0.000 meV
- 500.0 eV
- 1.000 keV
- 10.00 keV
- 100.0 keV
- 500.0 keV
- 750.0 keV
- 1.000 MeV
- 1.250 MeV
- 1.500 MeV
- 1.750 MeV

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Fission
Energy Distributions

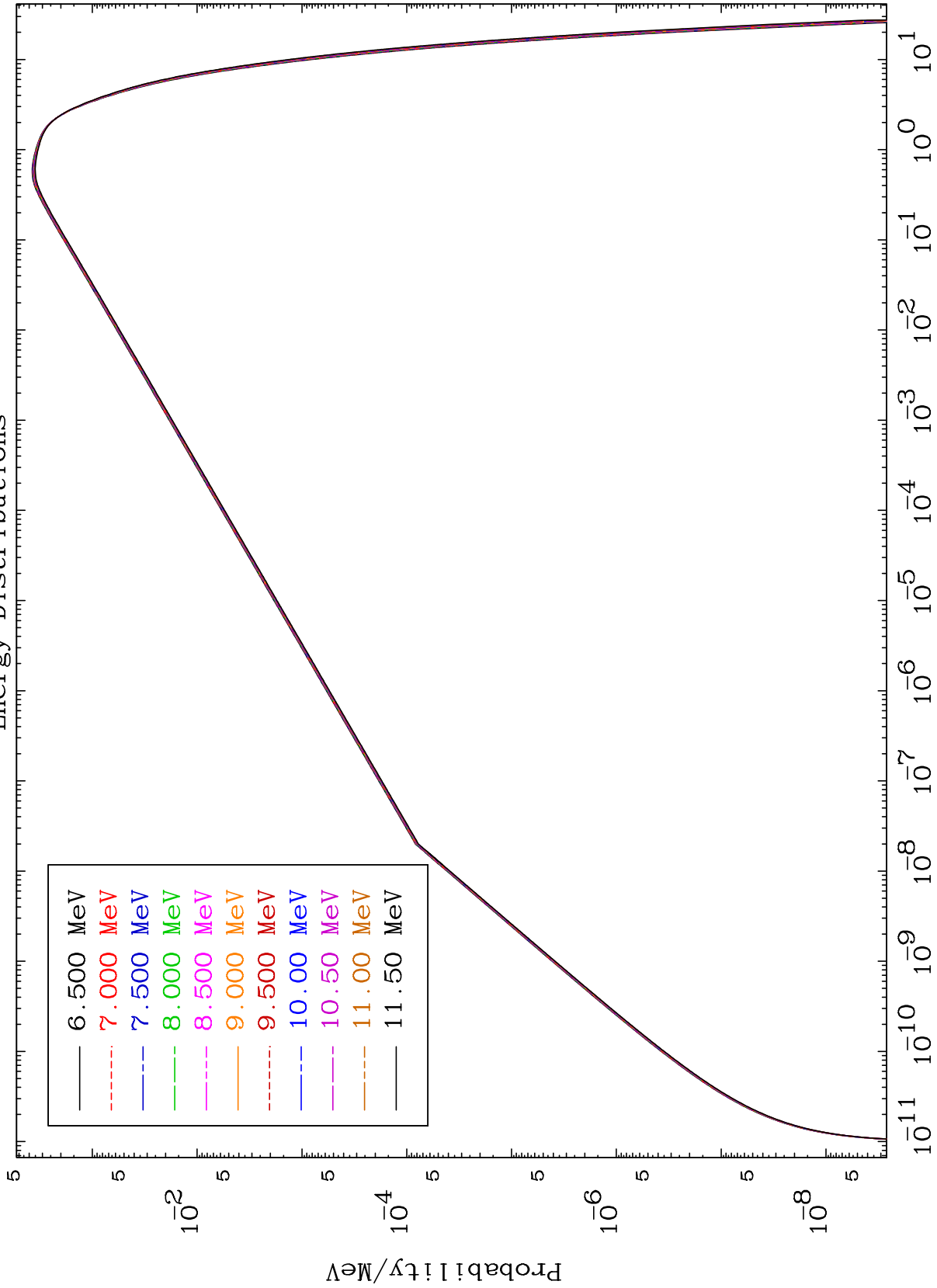
101-Md-247



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Fission
Energy Distributions

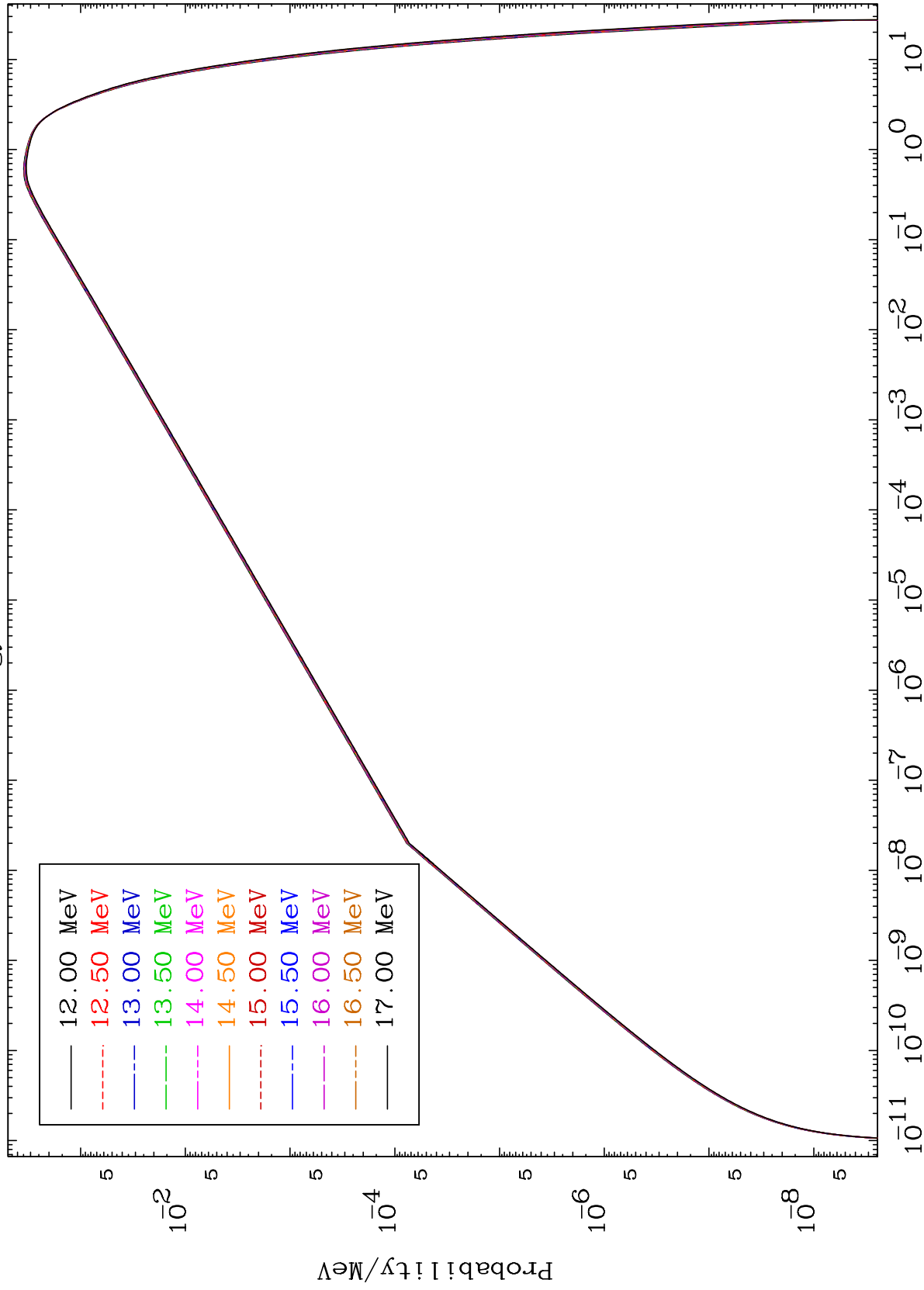
101-Md-247



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Fission
Energy Distributions

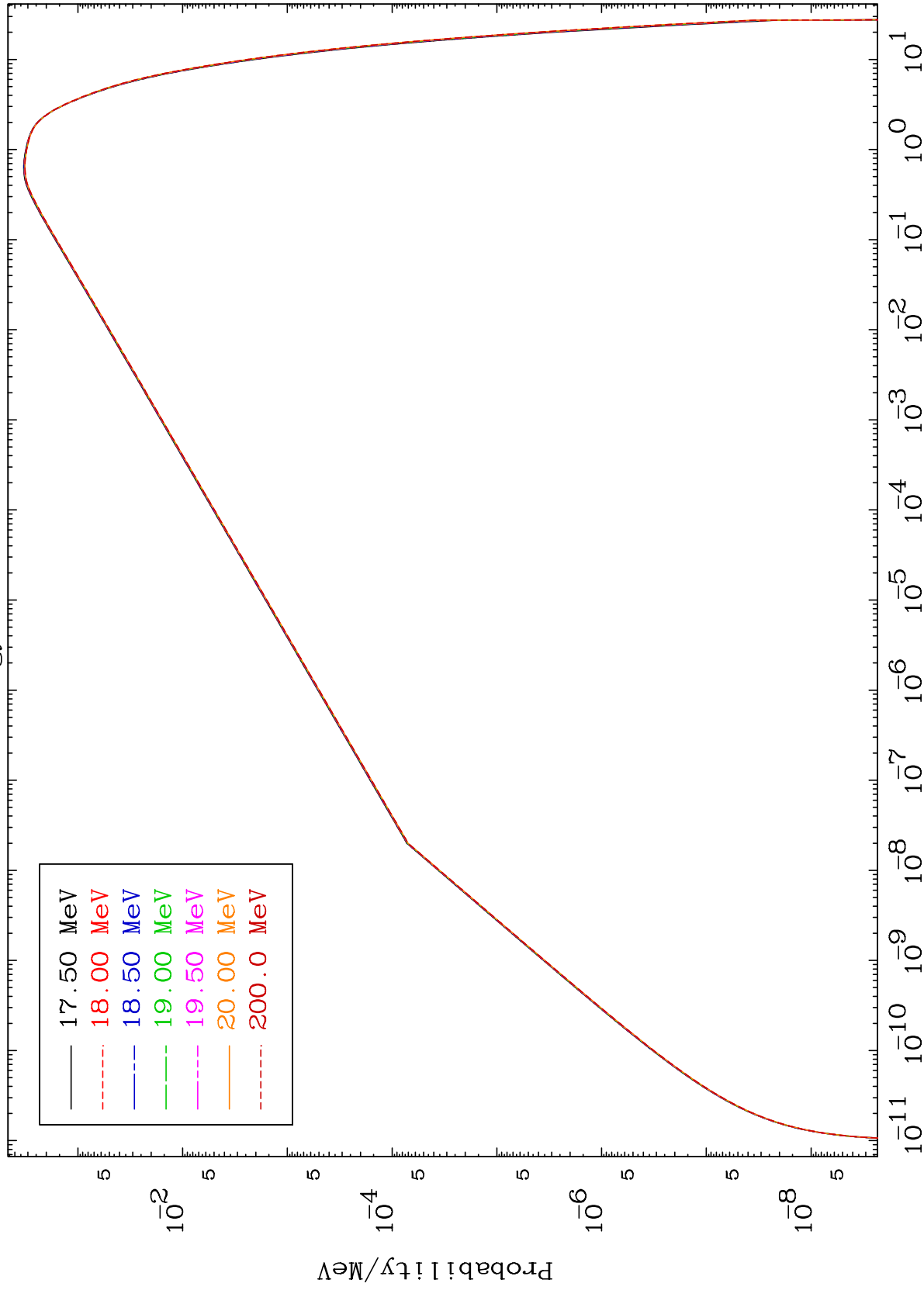
101-Md-247



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Fission
Energy Distributions

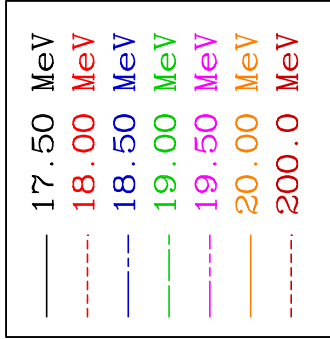
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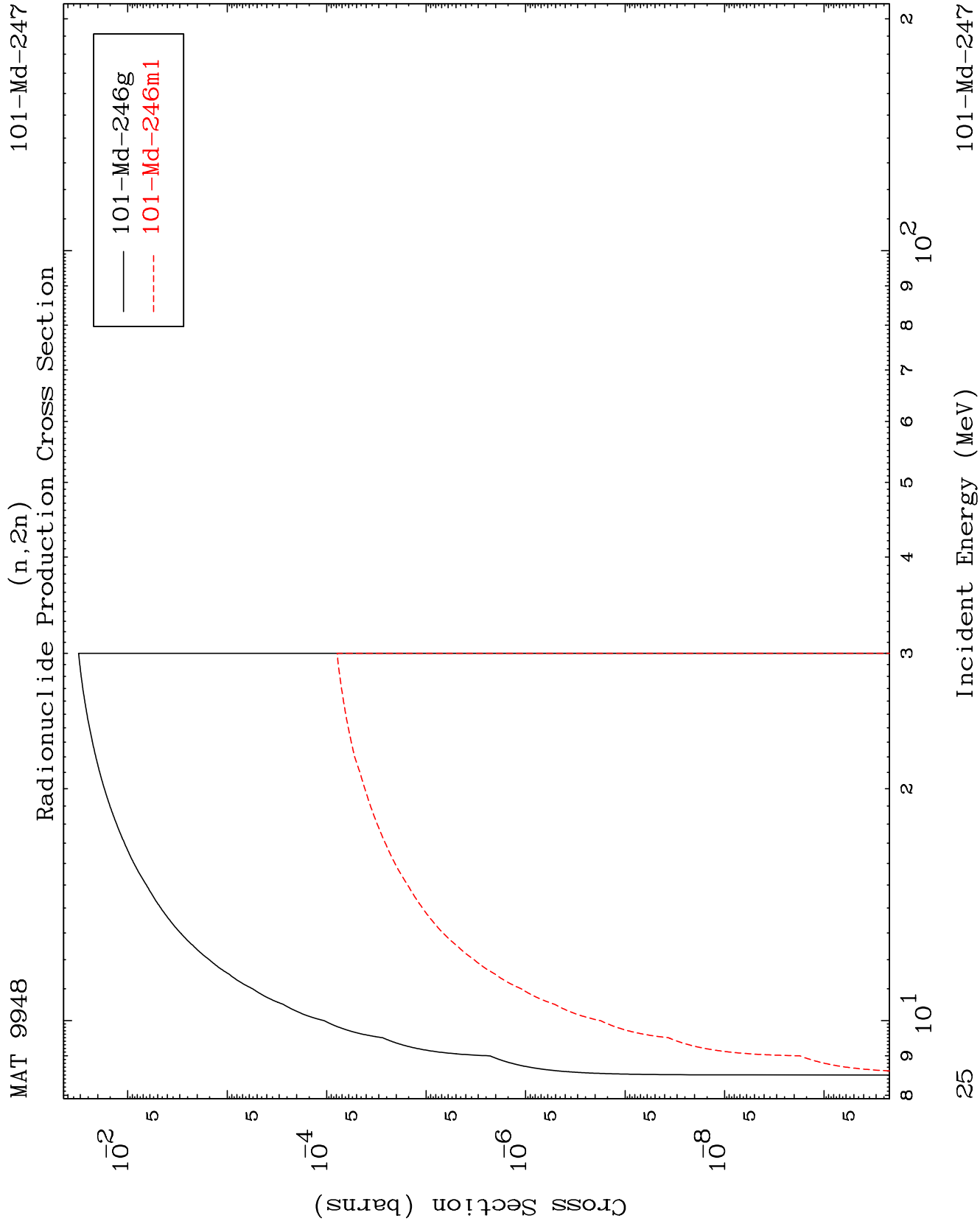


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Secondary Energy (MeV)

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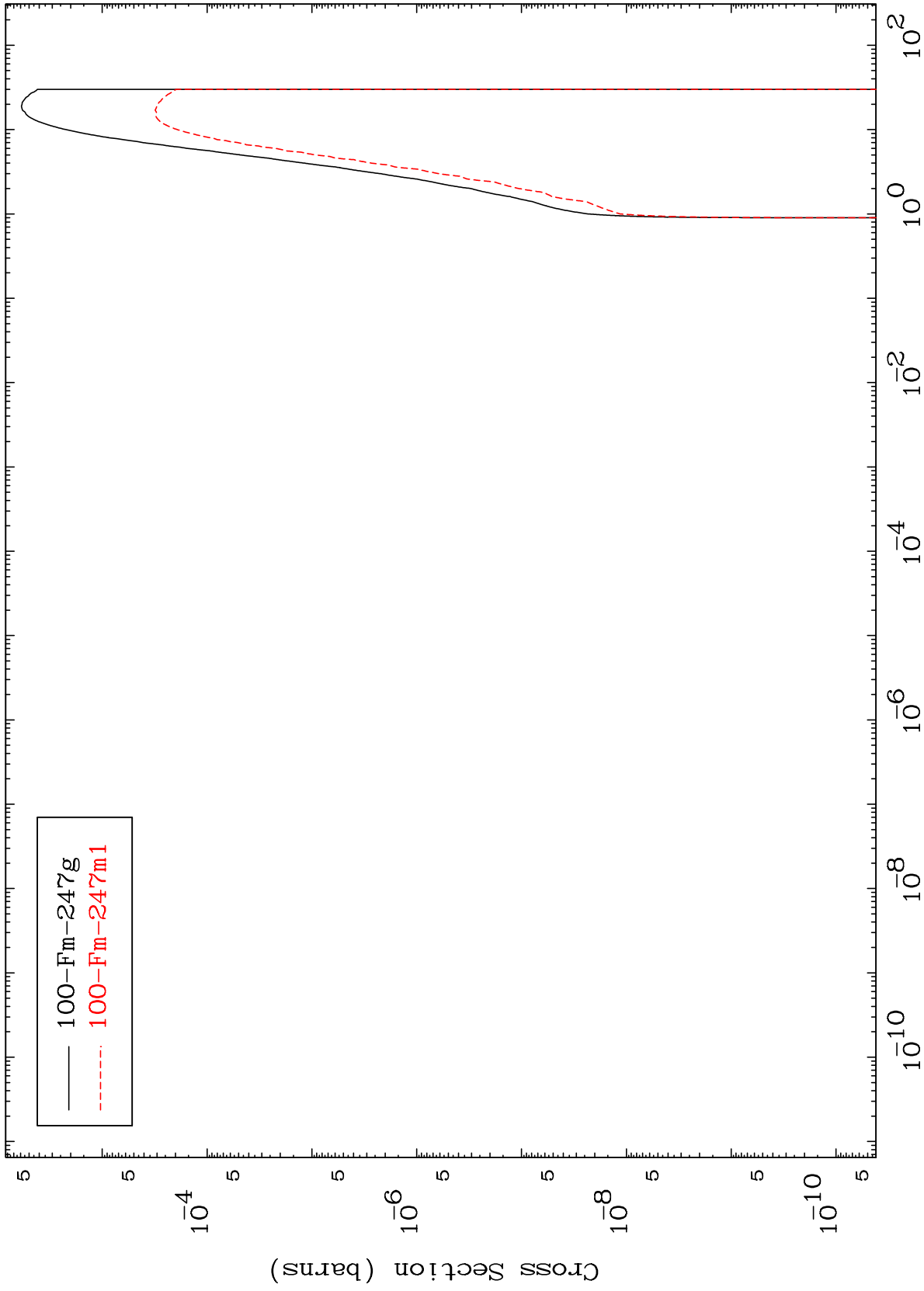




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(n,p)
Radionuclide Production Cross Section

101-Md-247



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Incident Energy (MeV)

101-Md-247