

Program EVALPLOT
(Version 2021-1)

by

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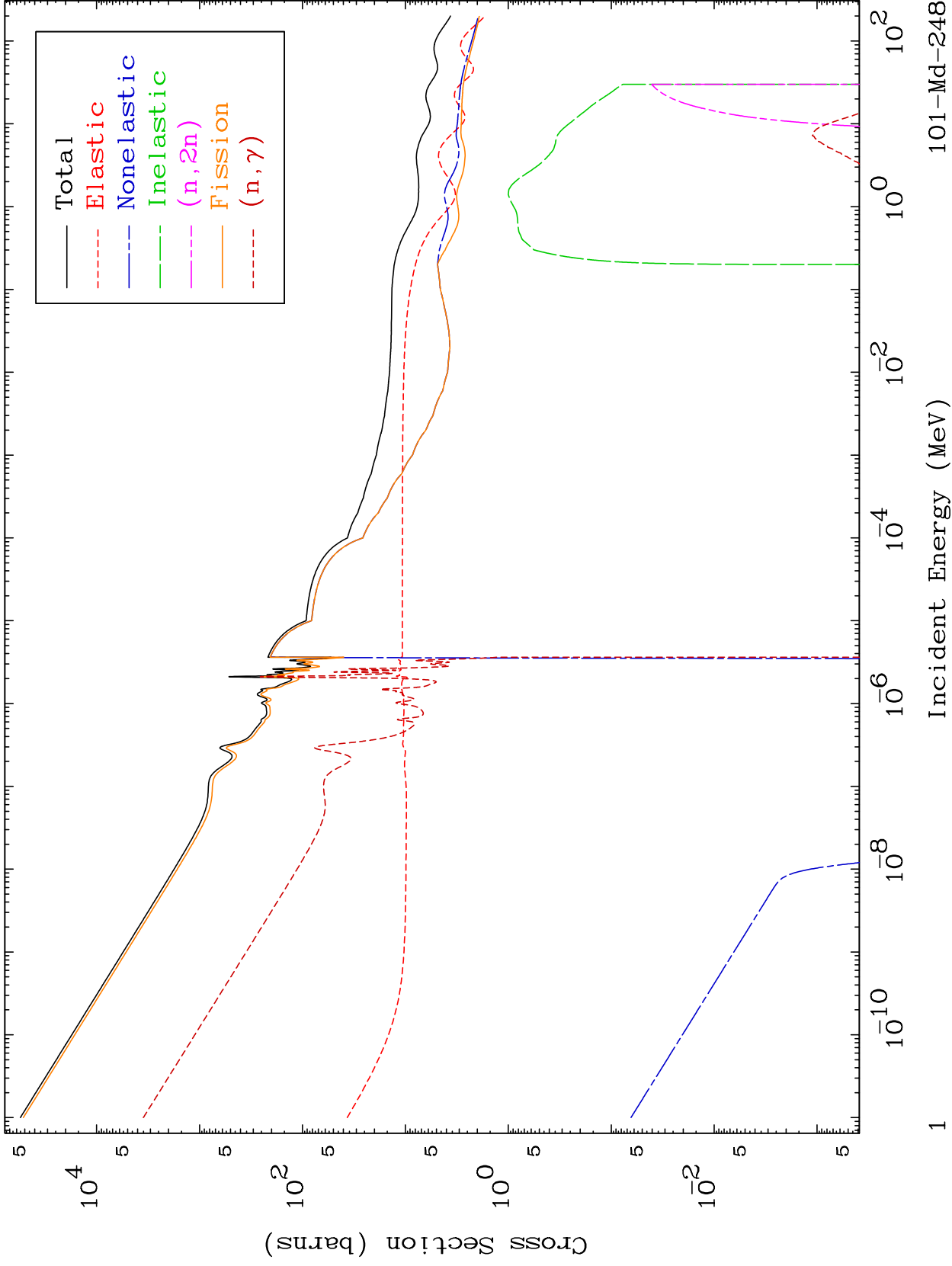
Web: redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9949

Neutron Major
293 Kelvin Cross Sections

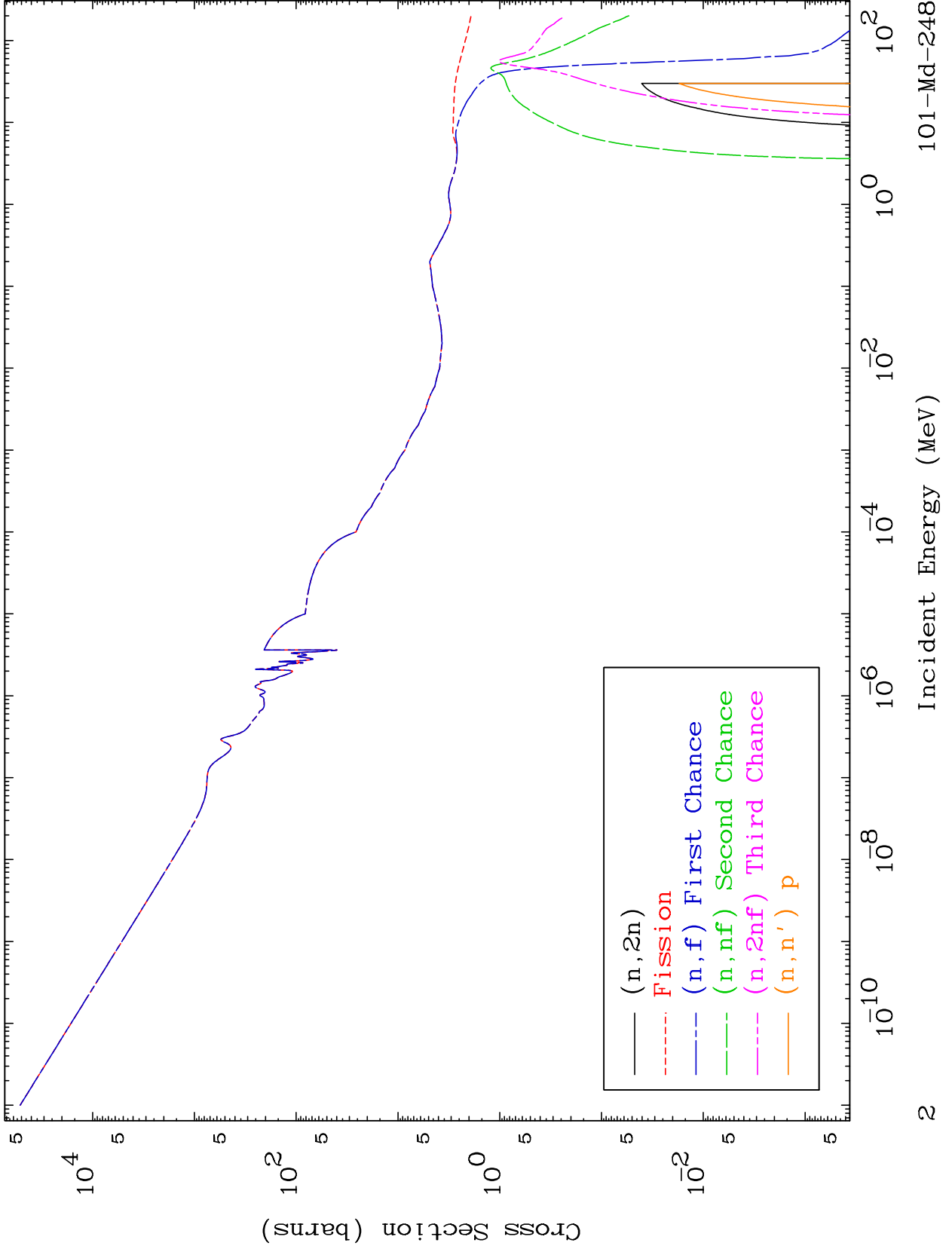
101-Md-248



MAT 9949

Neutron Absorption
293 Kelvin Cross Sections

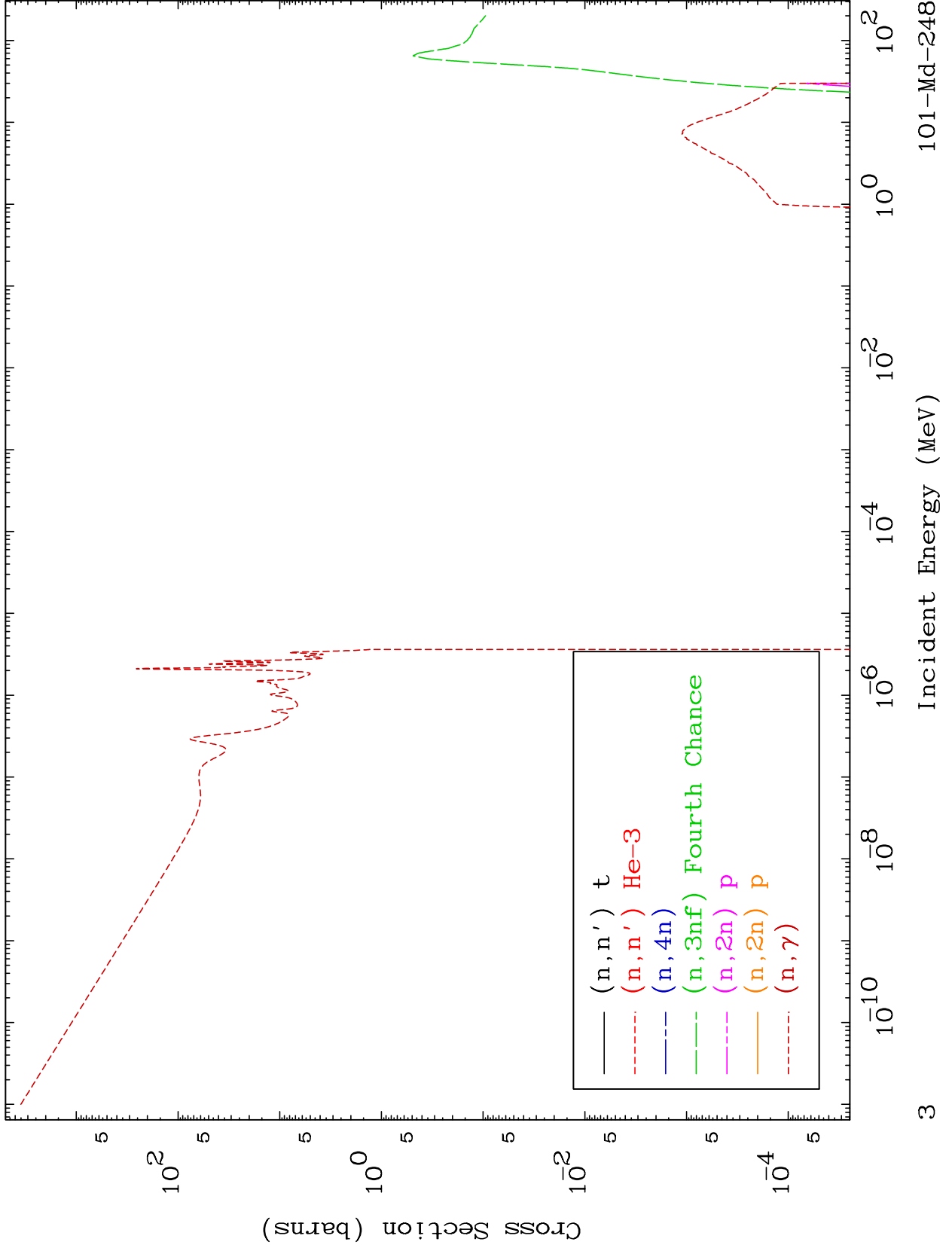
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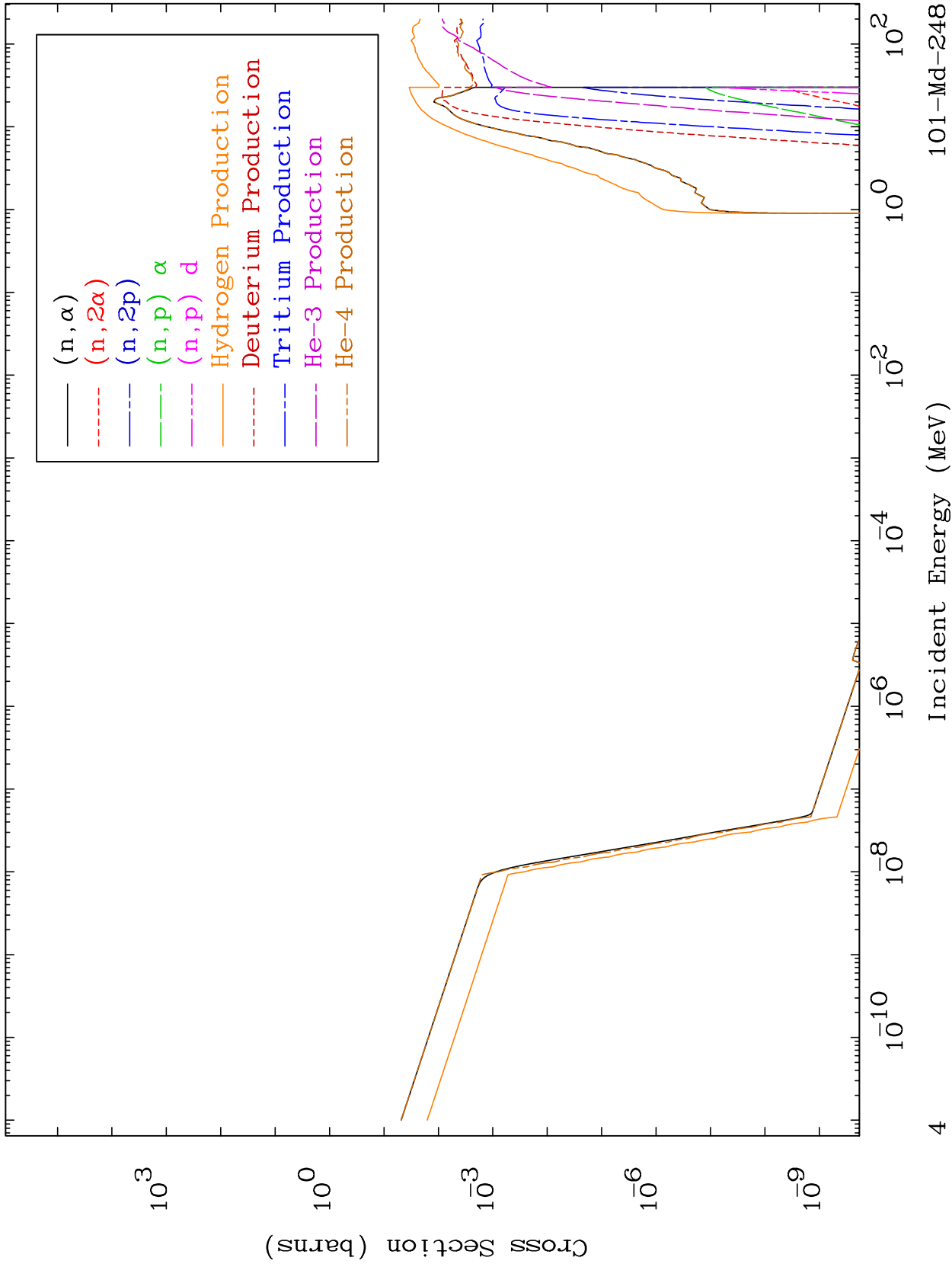


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Neutron Absorption
293 Kelvin Cross Sections

101-Md-248

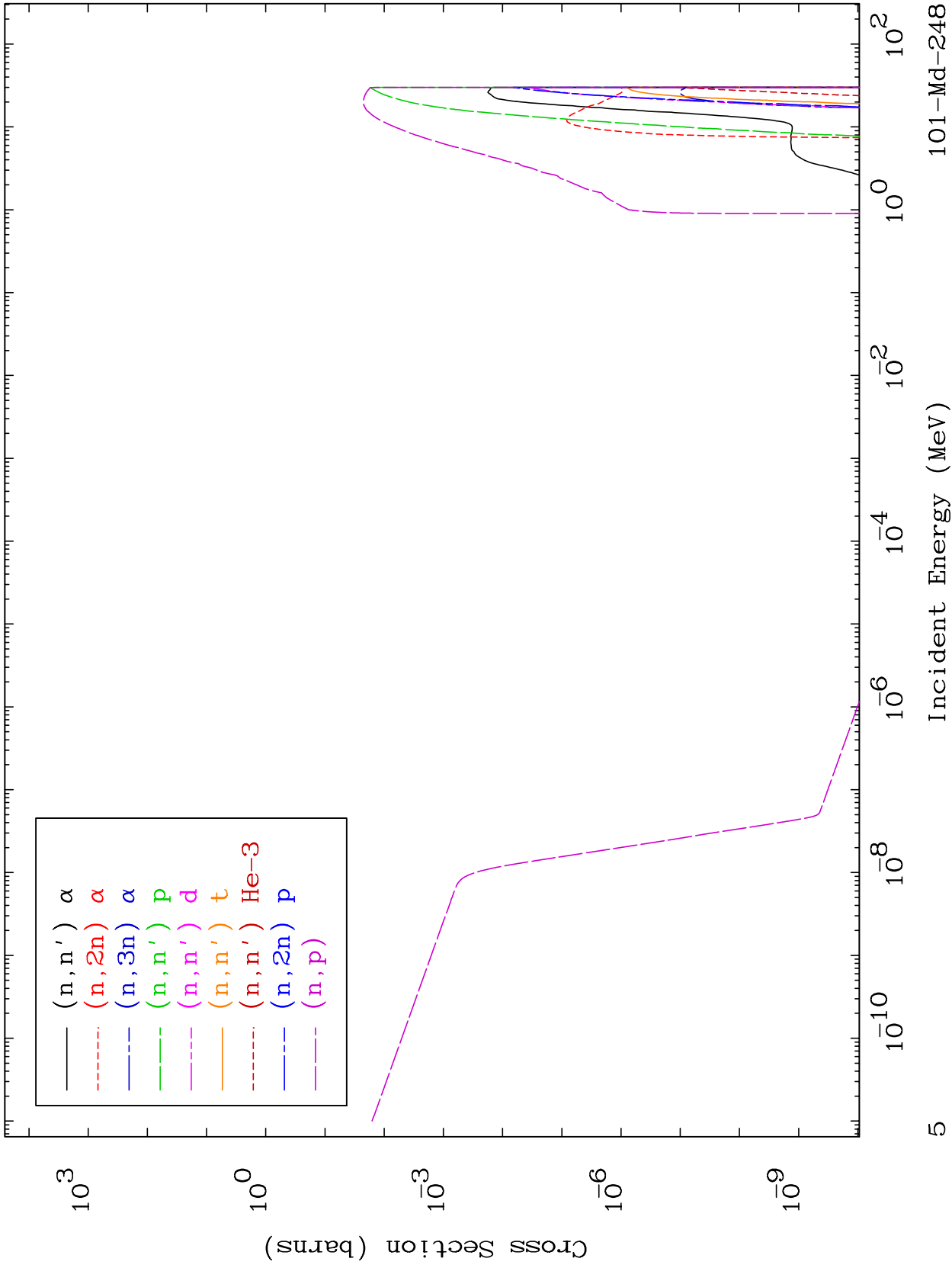




MAT 9949

Charged Particle
293 Kelvin Cross Sections

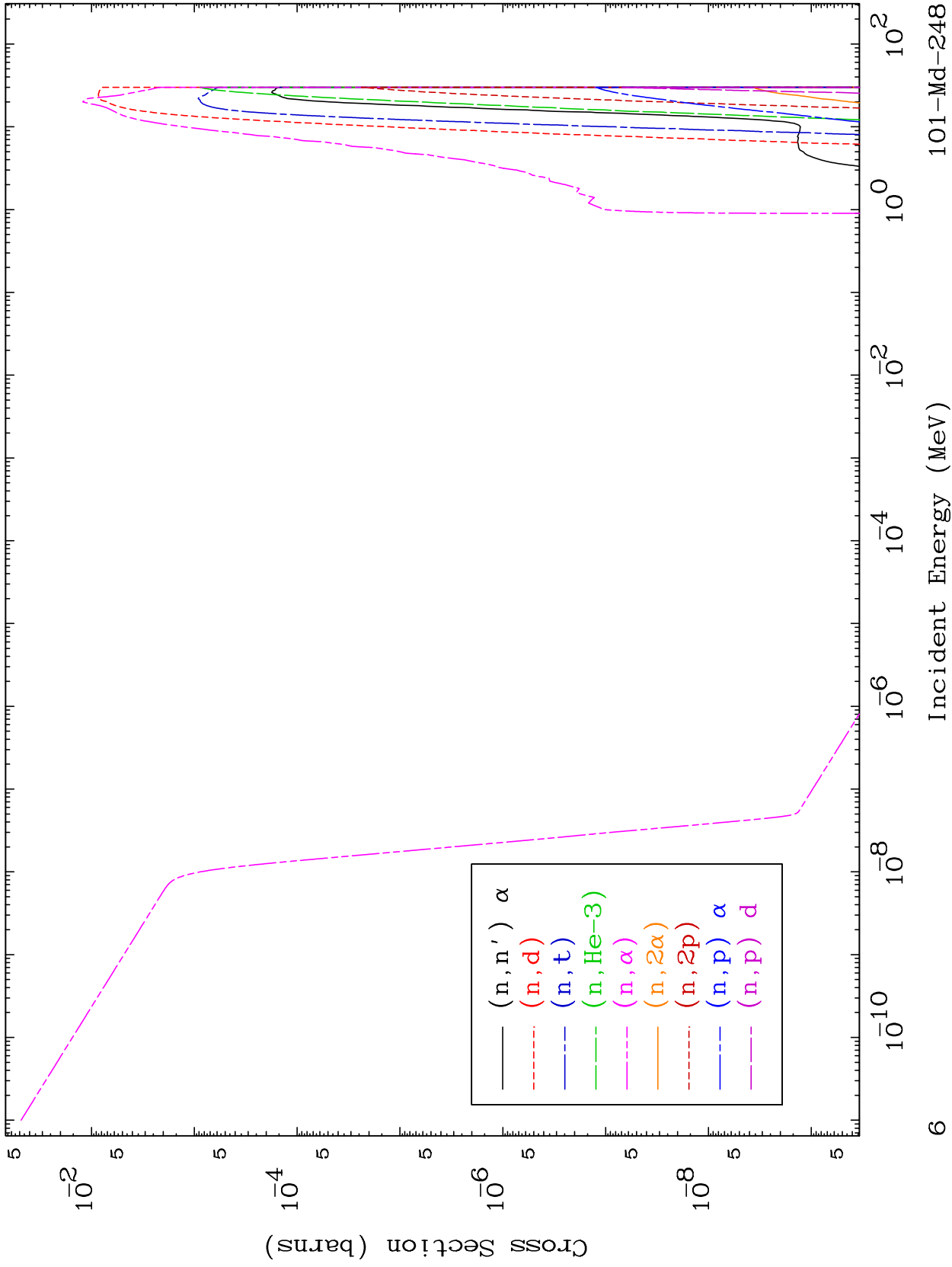
101-Md-248



MAT 9949

Charged Particle
293 Kelvin Cross Sections

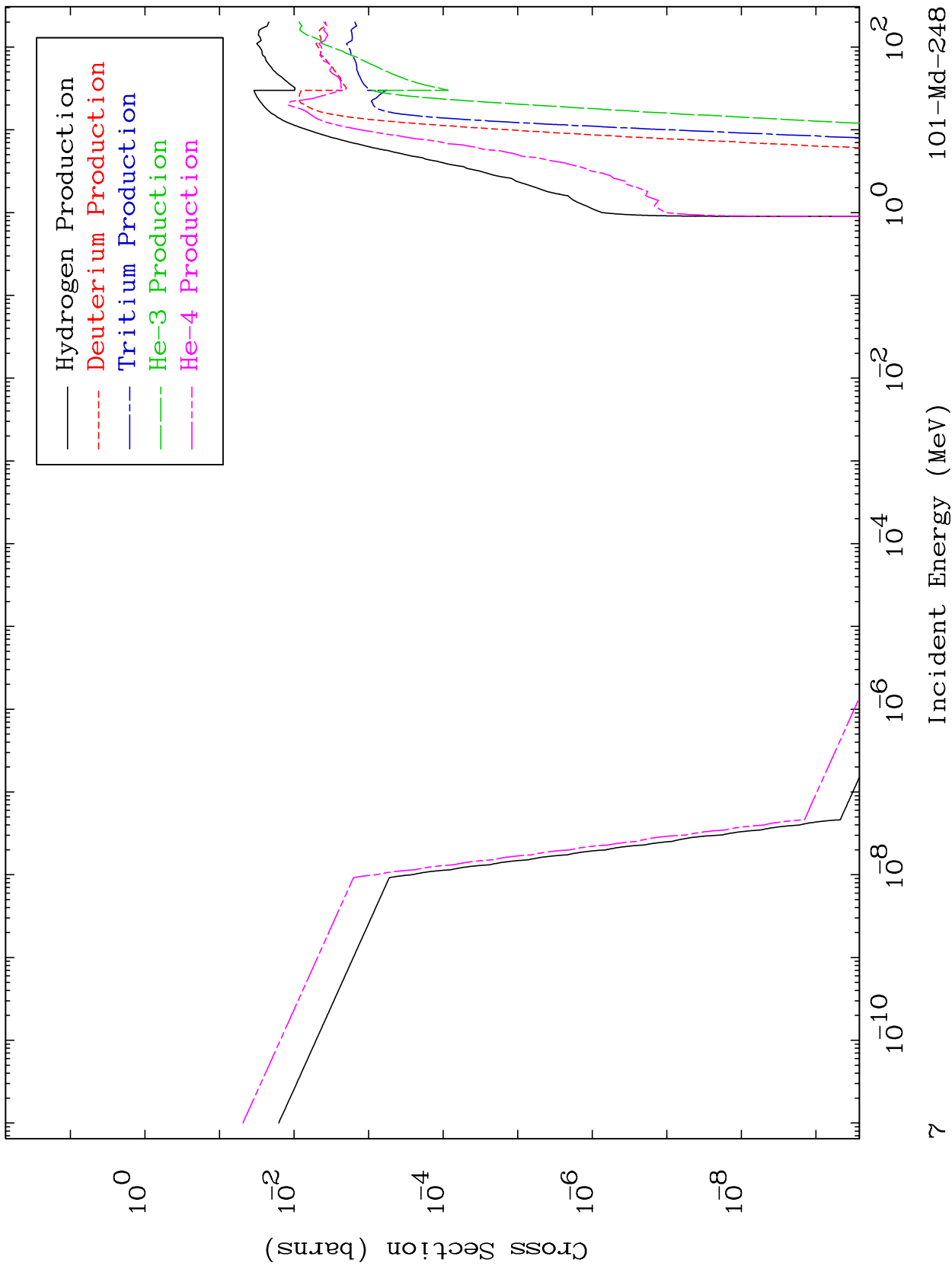
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Particle Production
293 Kelvin Cross Sections

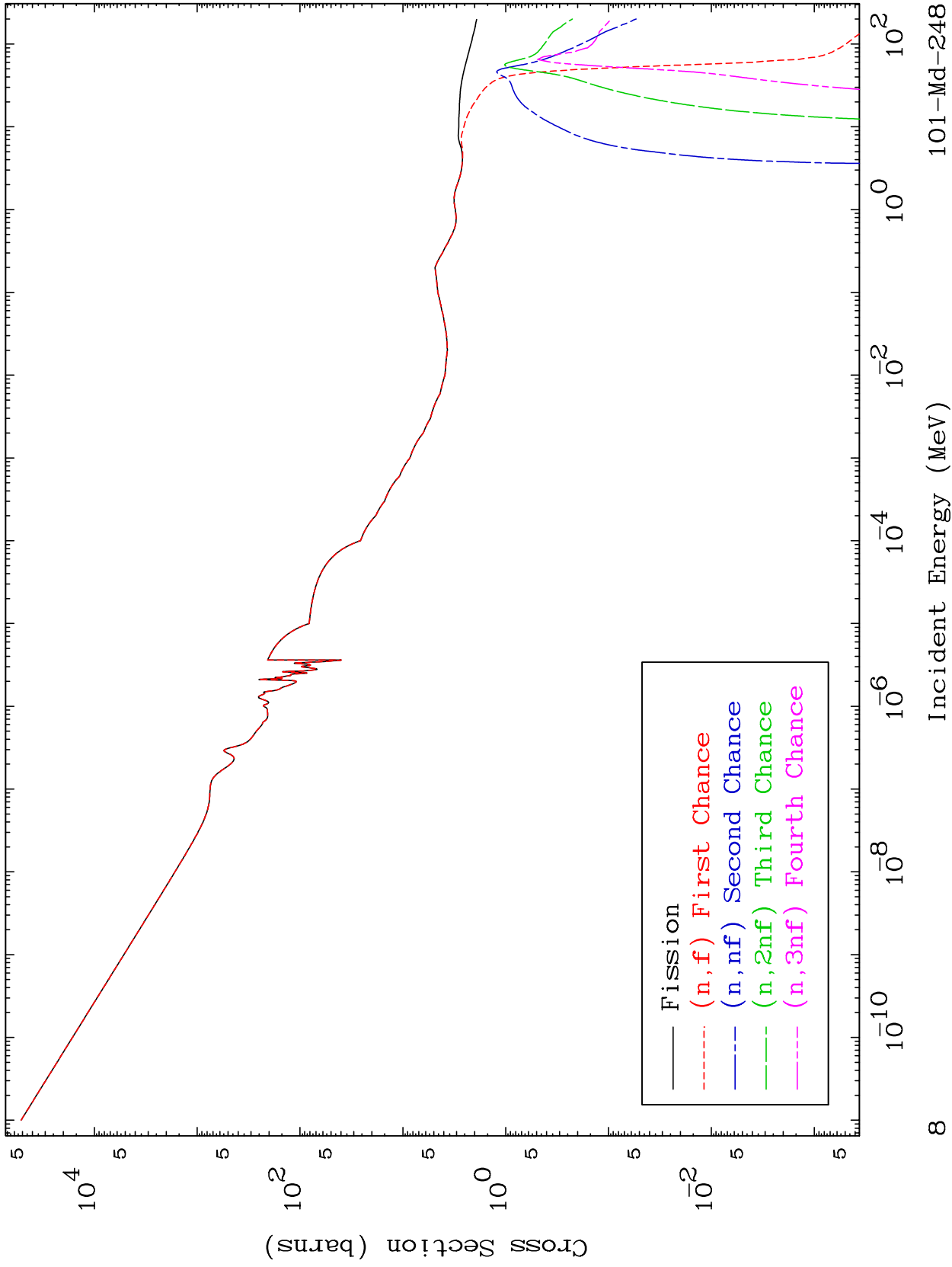
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Fission
293 Kelvin Cross Sections

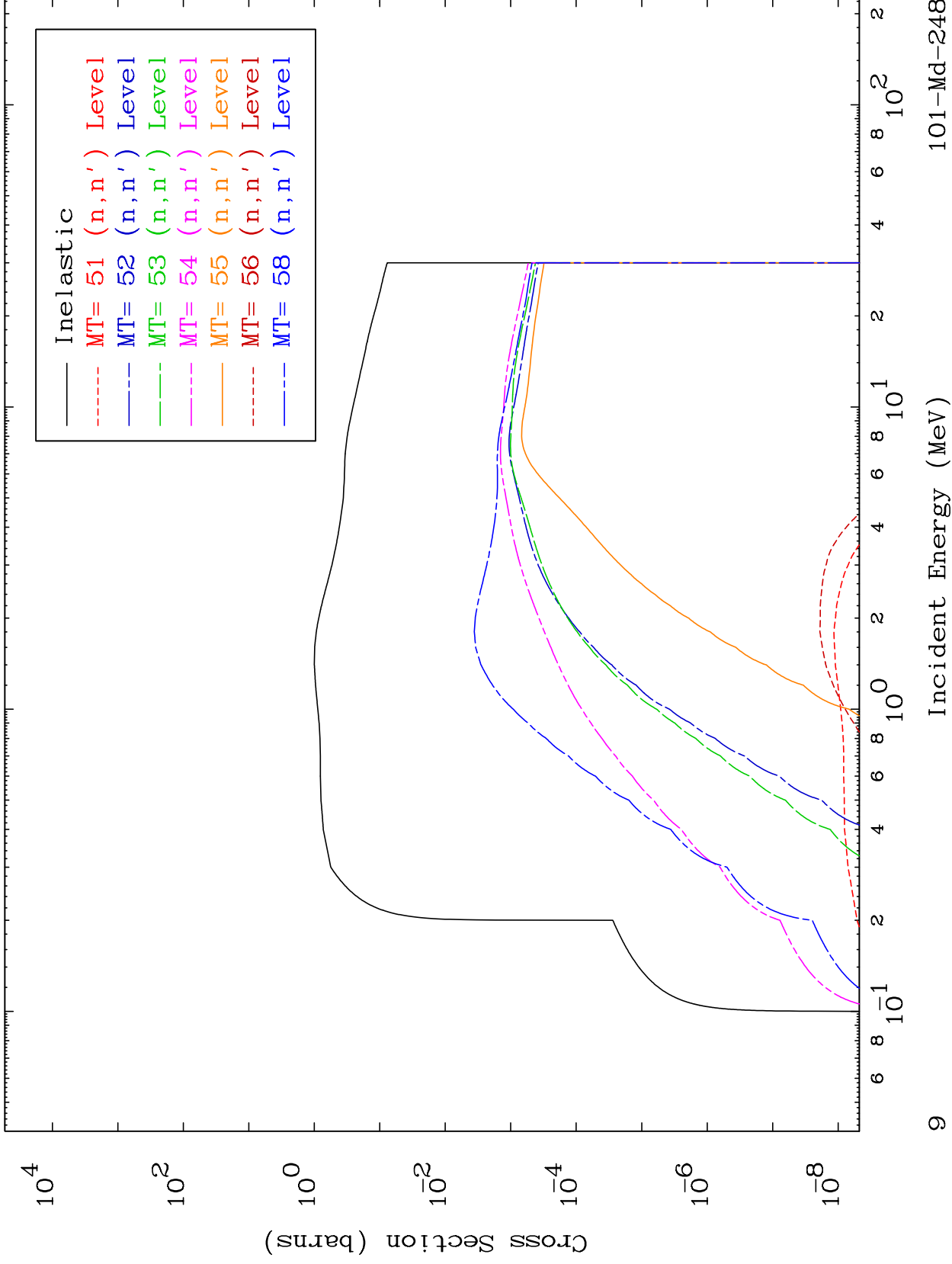
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MAT 9949

(n,n') Levels
293 Kelvin Cross Sections

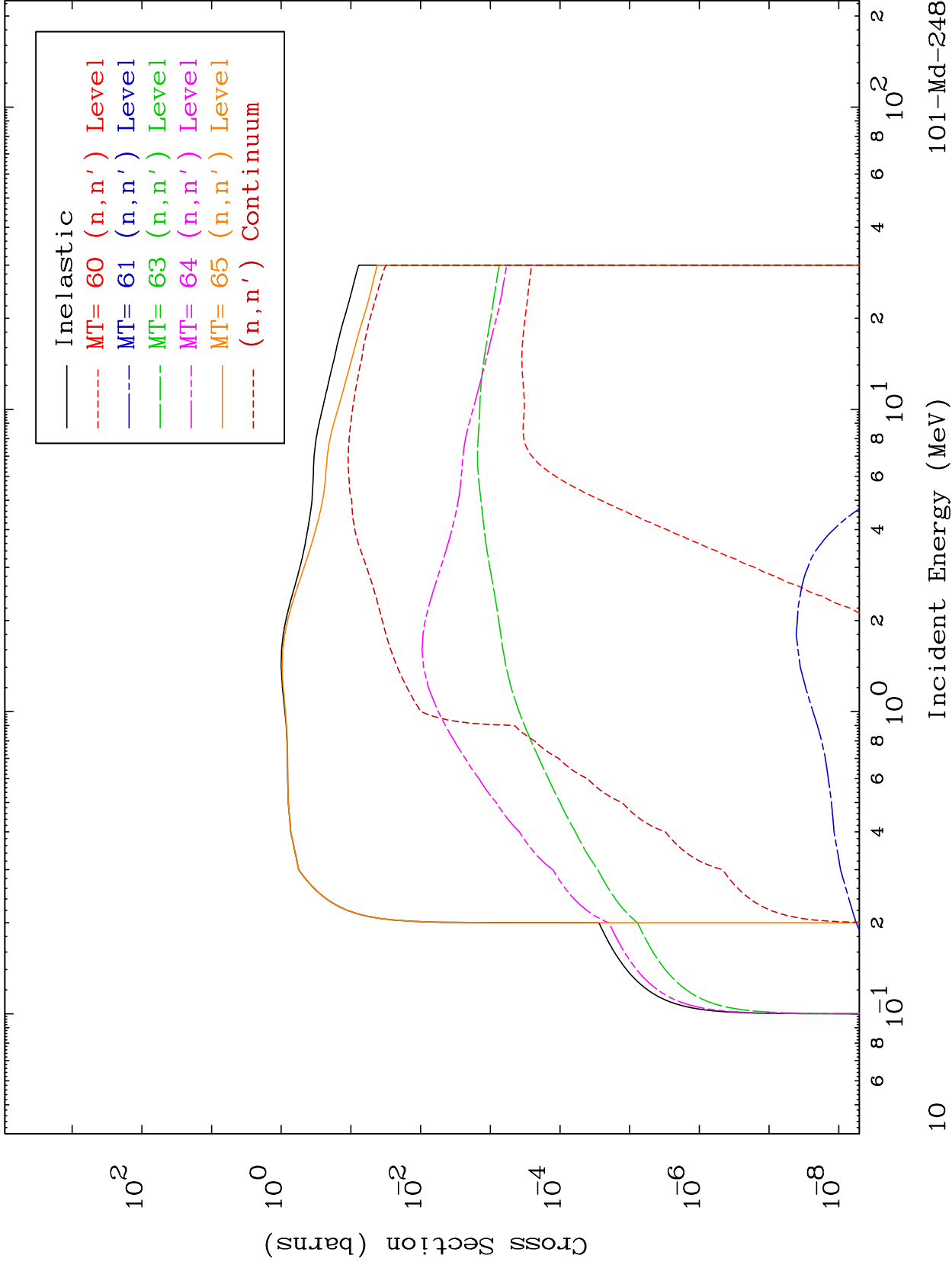
101-Md-248



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(n,n') Levels
293 Kelvin Cross Sections

101-Md-248



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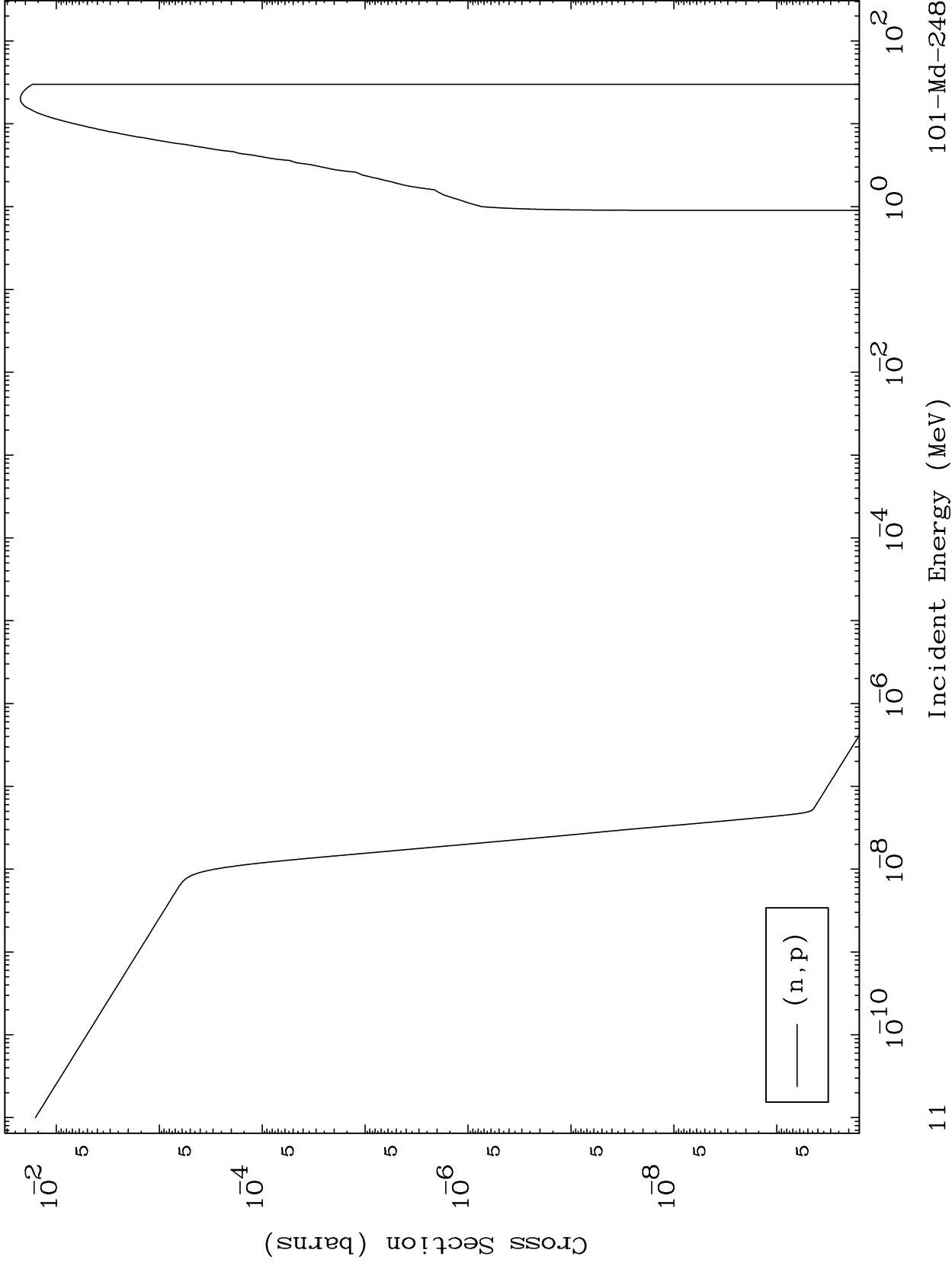
Incident Energy (MeV)

101-Md-248

MAT 9949

(n,p) Levels
293 Kelvin Cross Sections

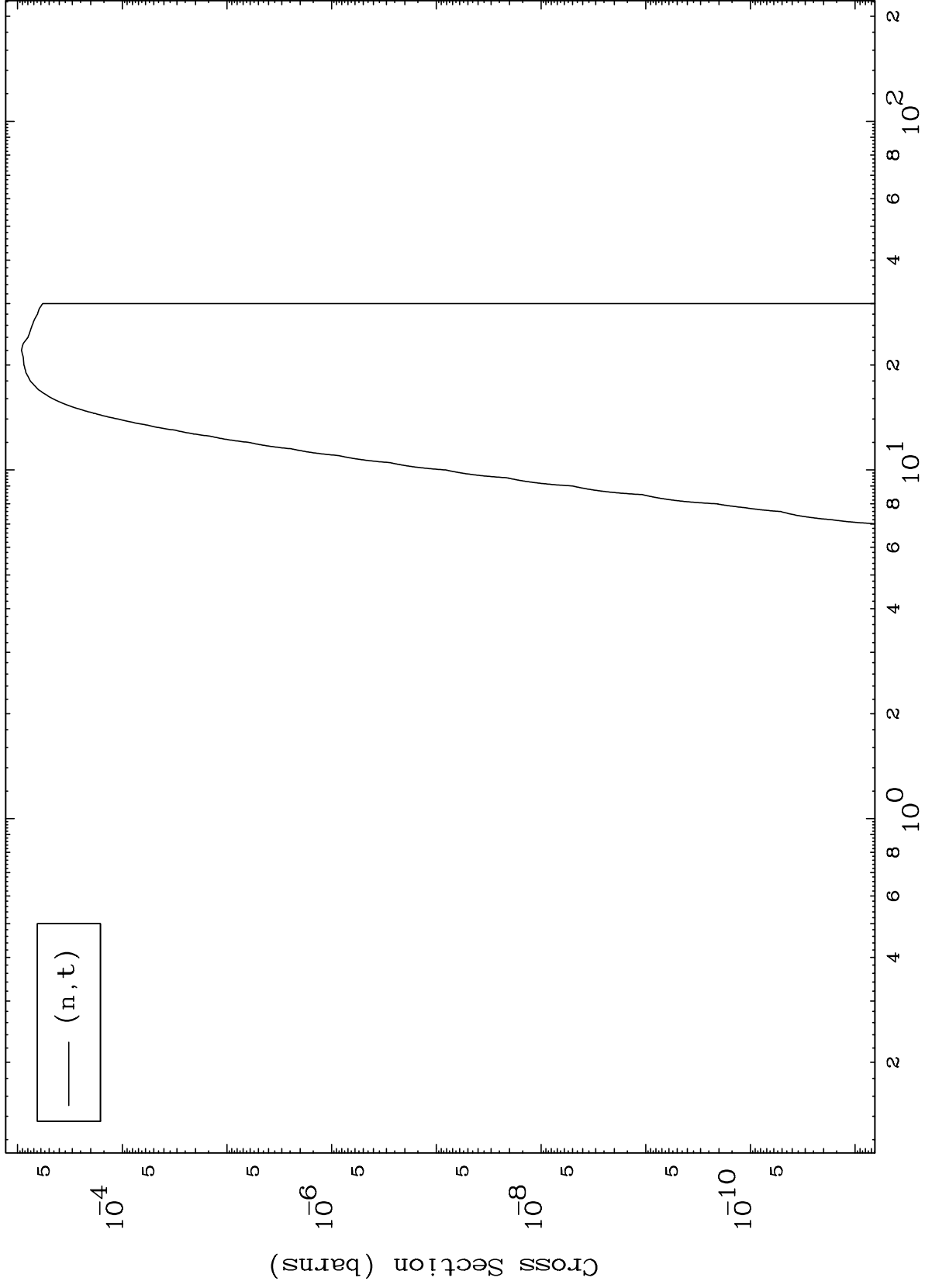
101-Md-248



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(n,t) Levels
293 Kelvin Cross Sections

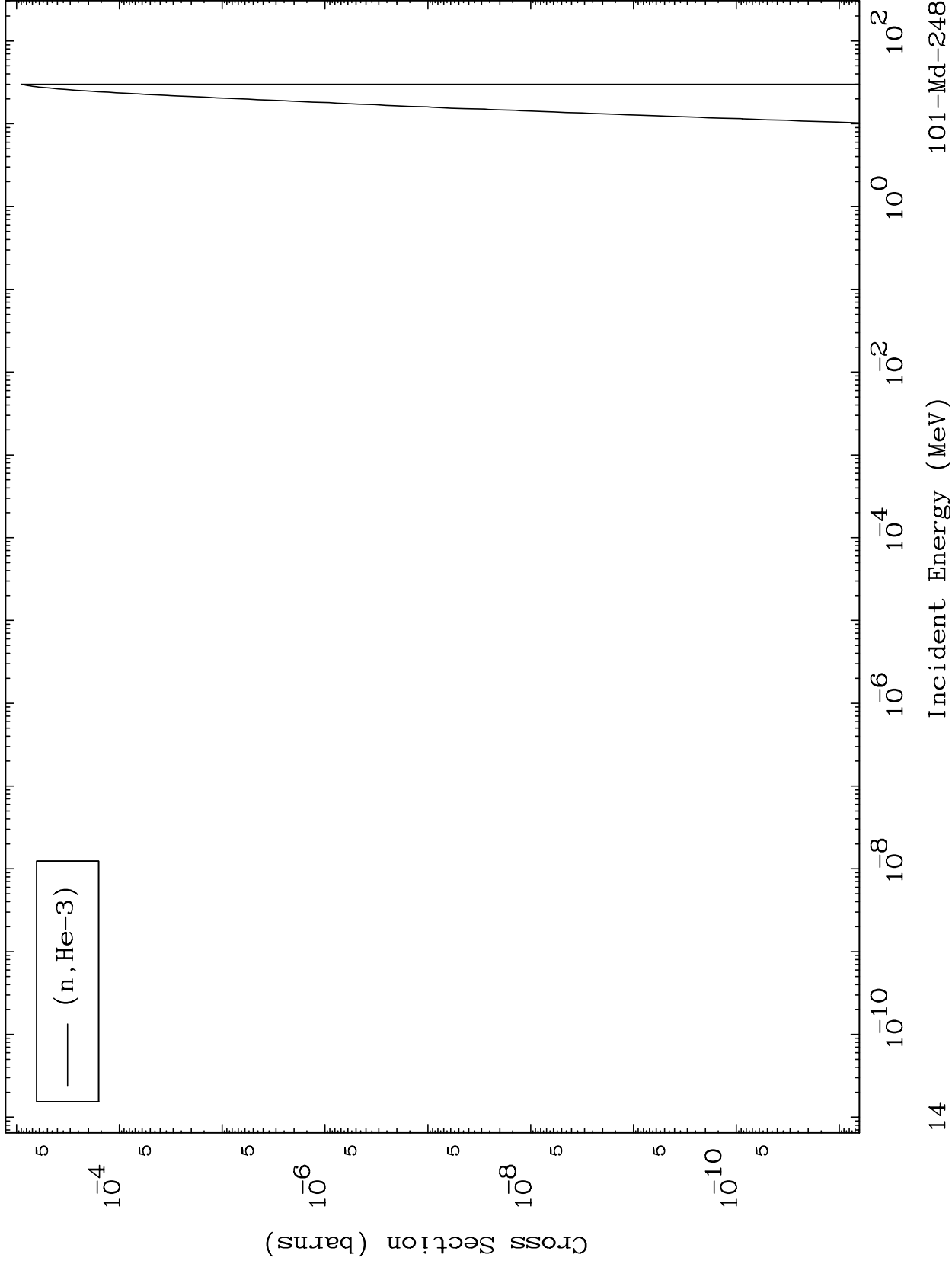
101-Md-248



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(n,He3) Levels
293 Kelvin Cross Sections

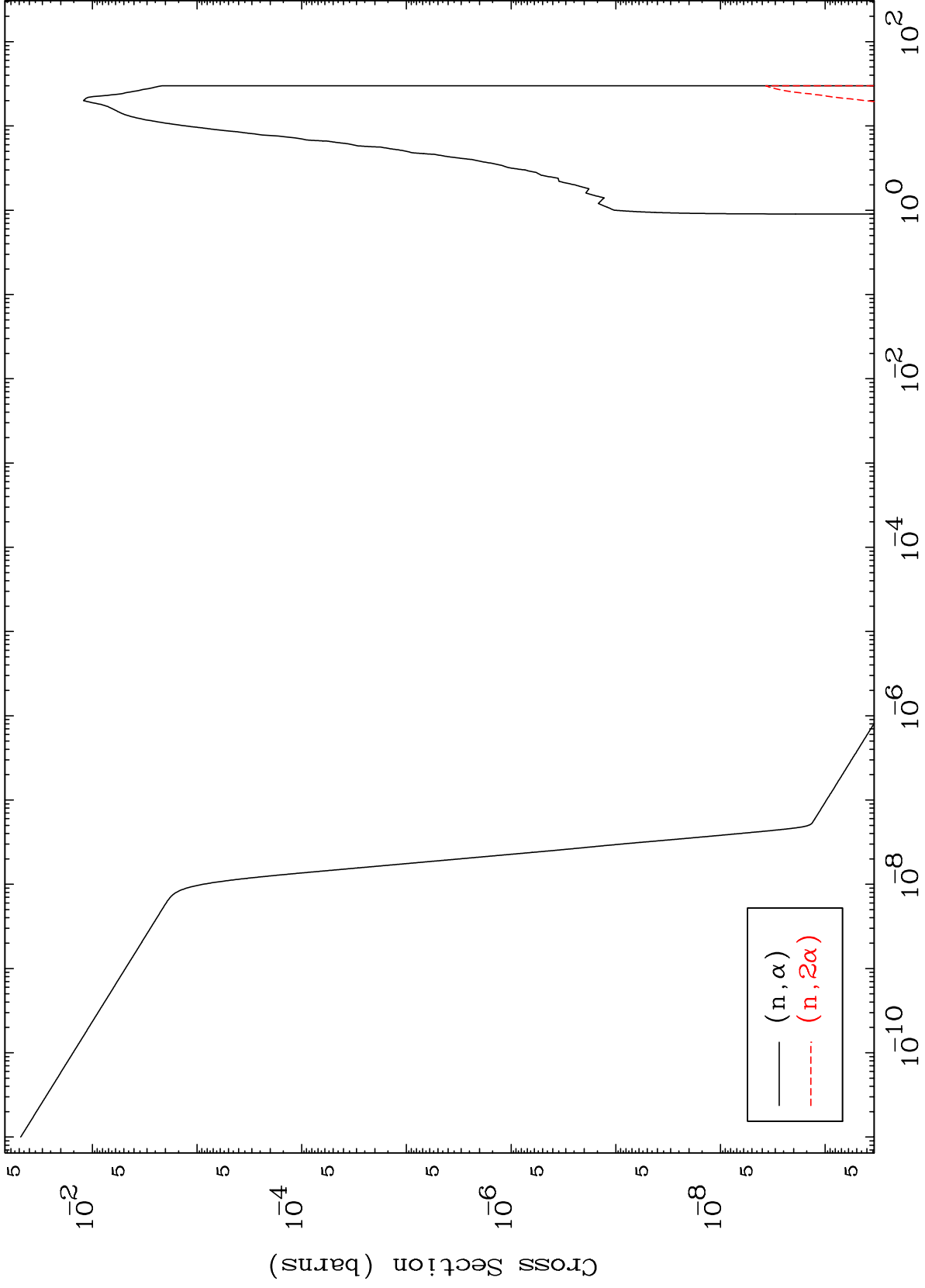
101-Md-248



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(n, α) Levels
293 Kelvin Cross Sections

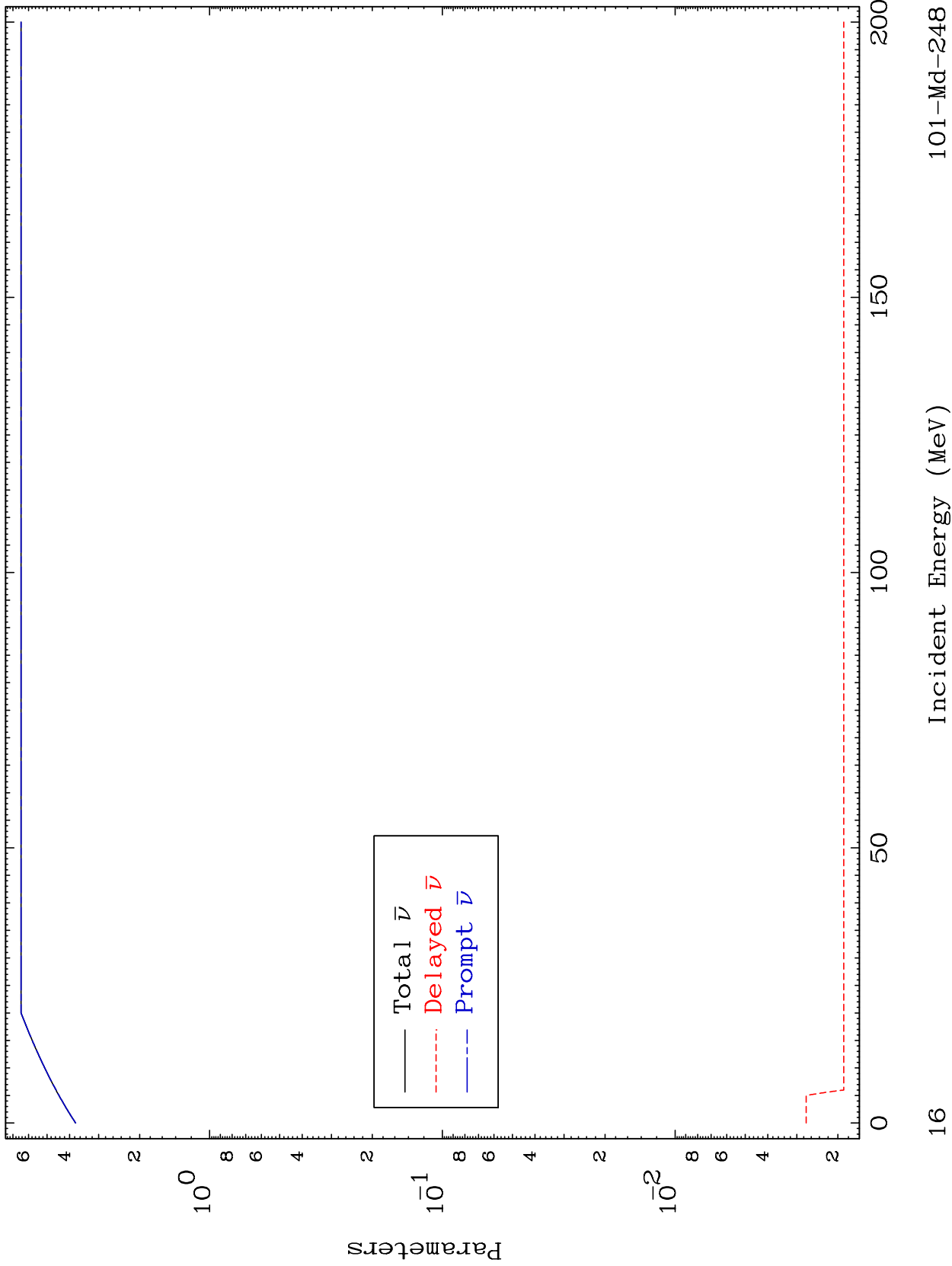
101-Md-248



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Energy Release
Parameters

101-Md-248



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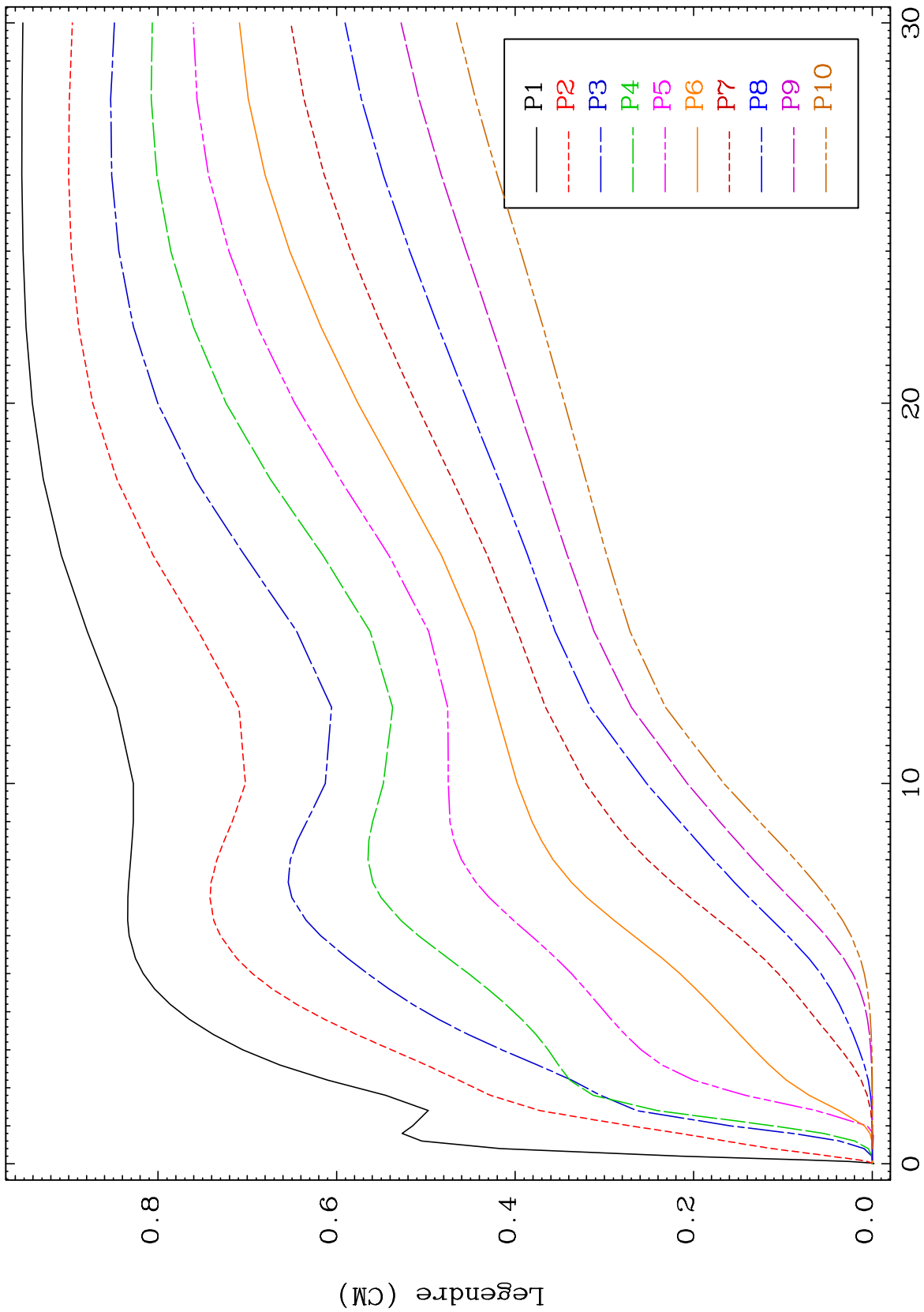
Incident Energy (MeV)

101-Md-248

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101-Md-248

Elastic Legendre Coefficients



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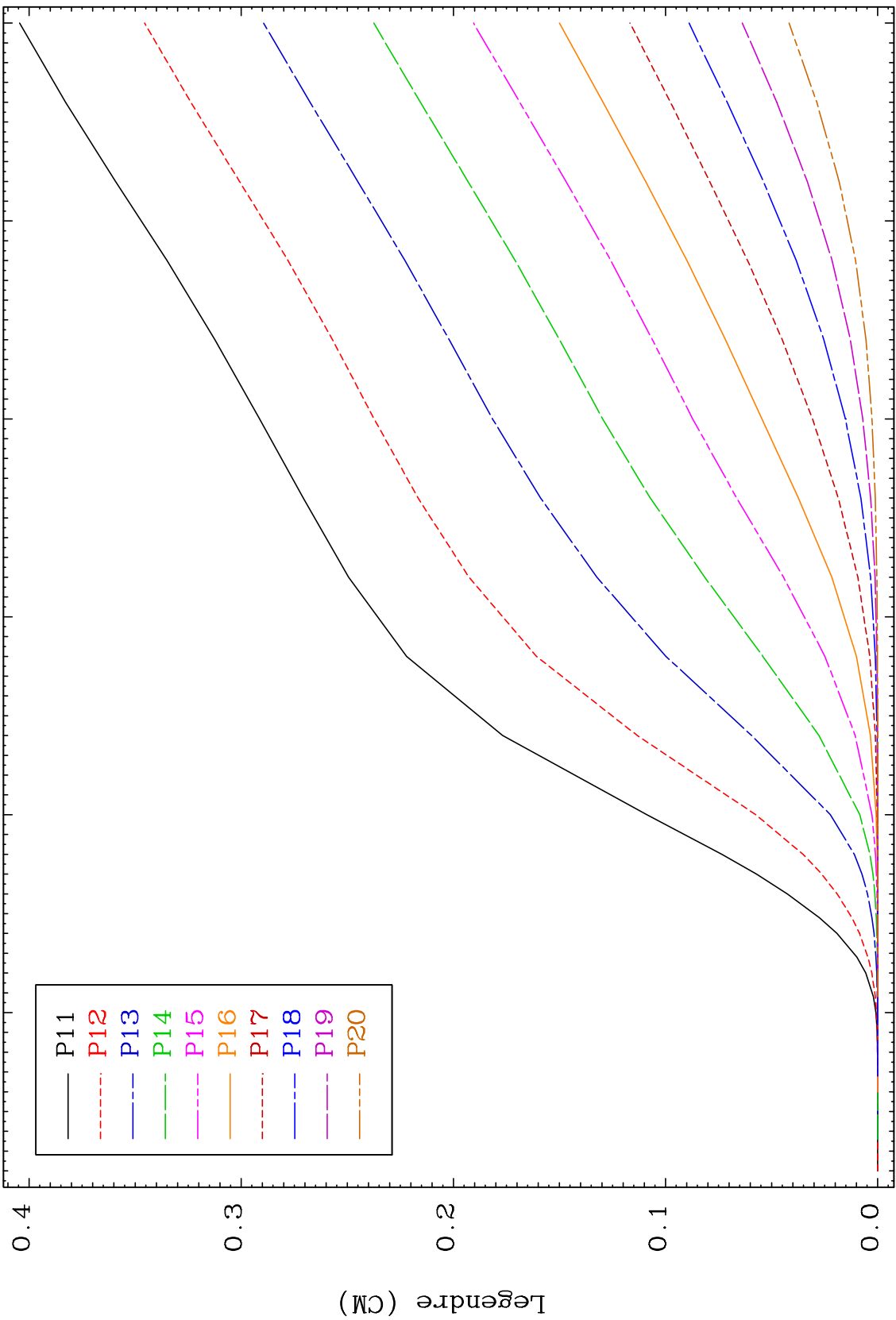
Incident Energy (MeV)

101-Md-248

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Elastic Legendre Coefficients

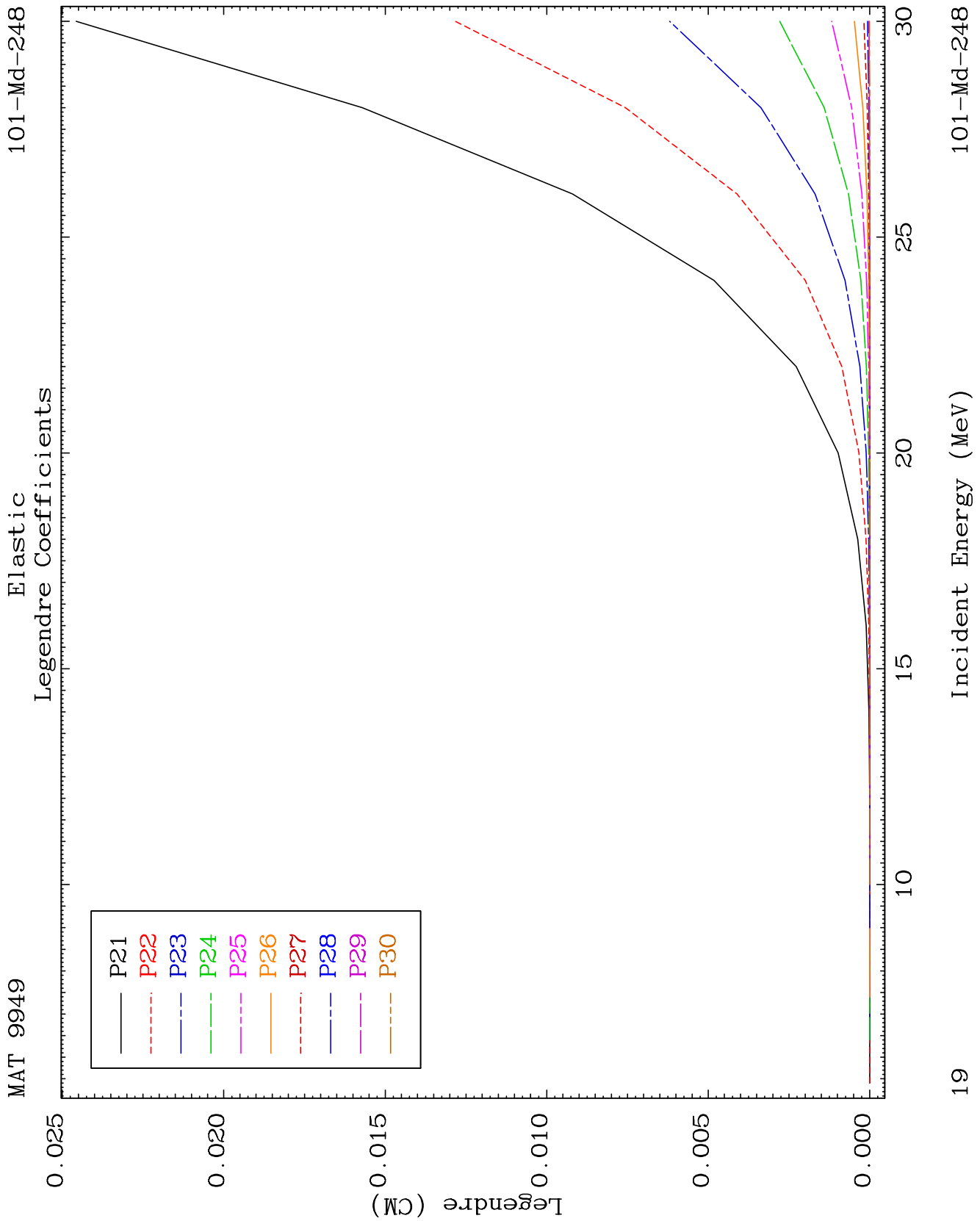
101-Md-248



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Incident Energy (MeV)

101-Md-248



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Elastic Legendre Coefficients

$^{101}\text{Md-248}$

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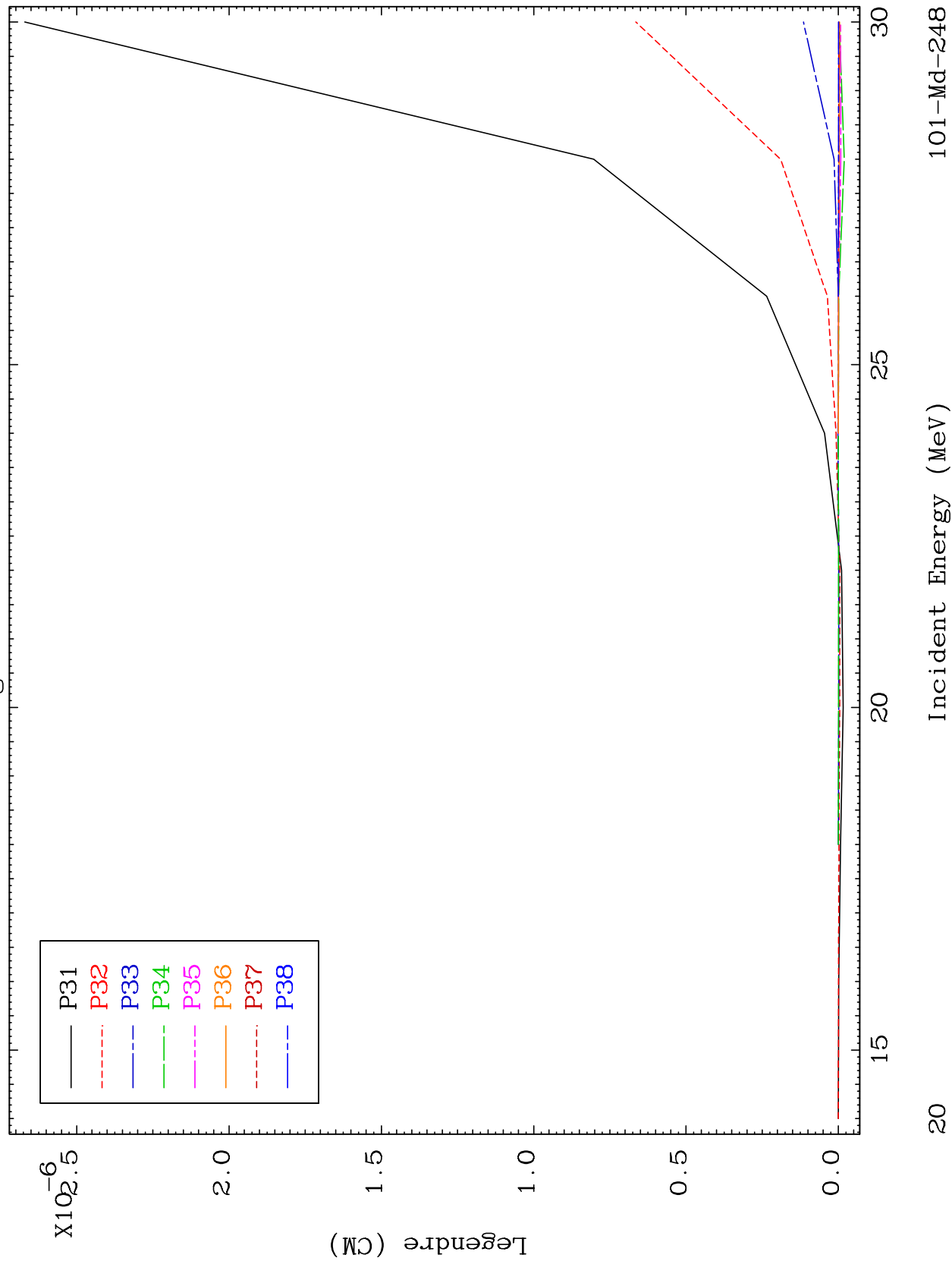
Incident Energy (MeV)

$^{101}\text{Md-248}$

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Elastic Legendre Coefficients

101-Md-248



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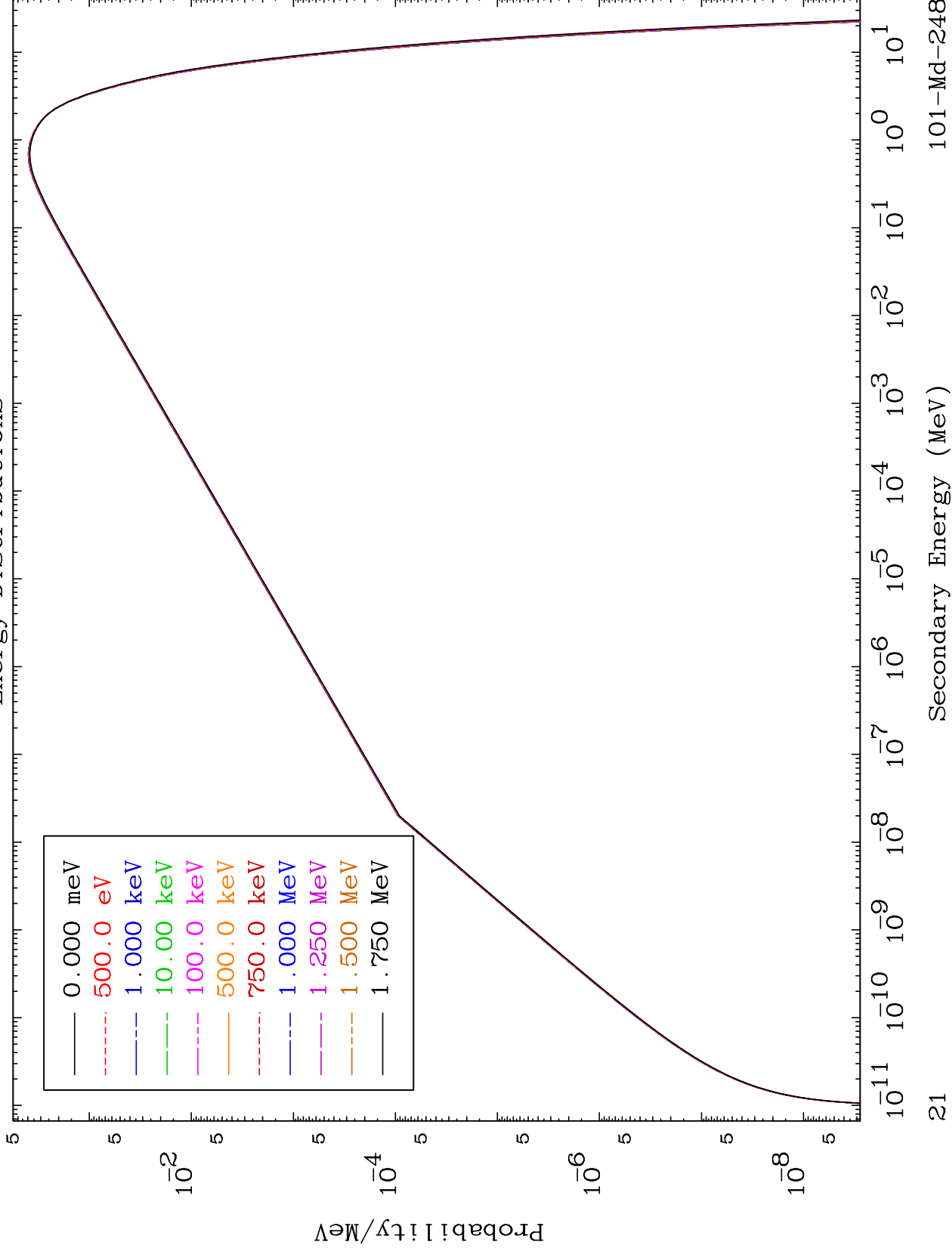
Incident Energy (MeV)

101-Md-248

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Fission Energy Distributions

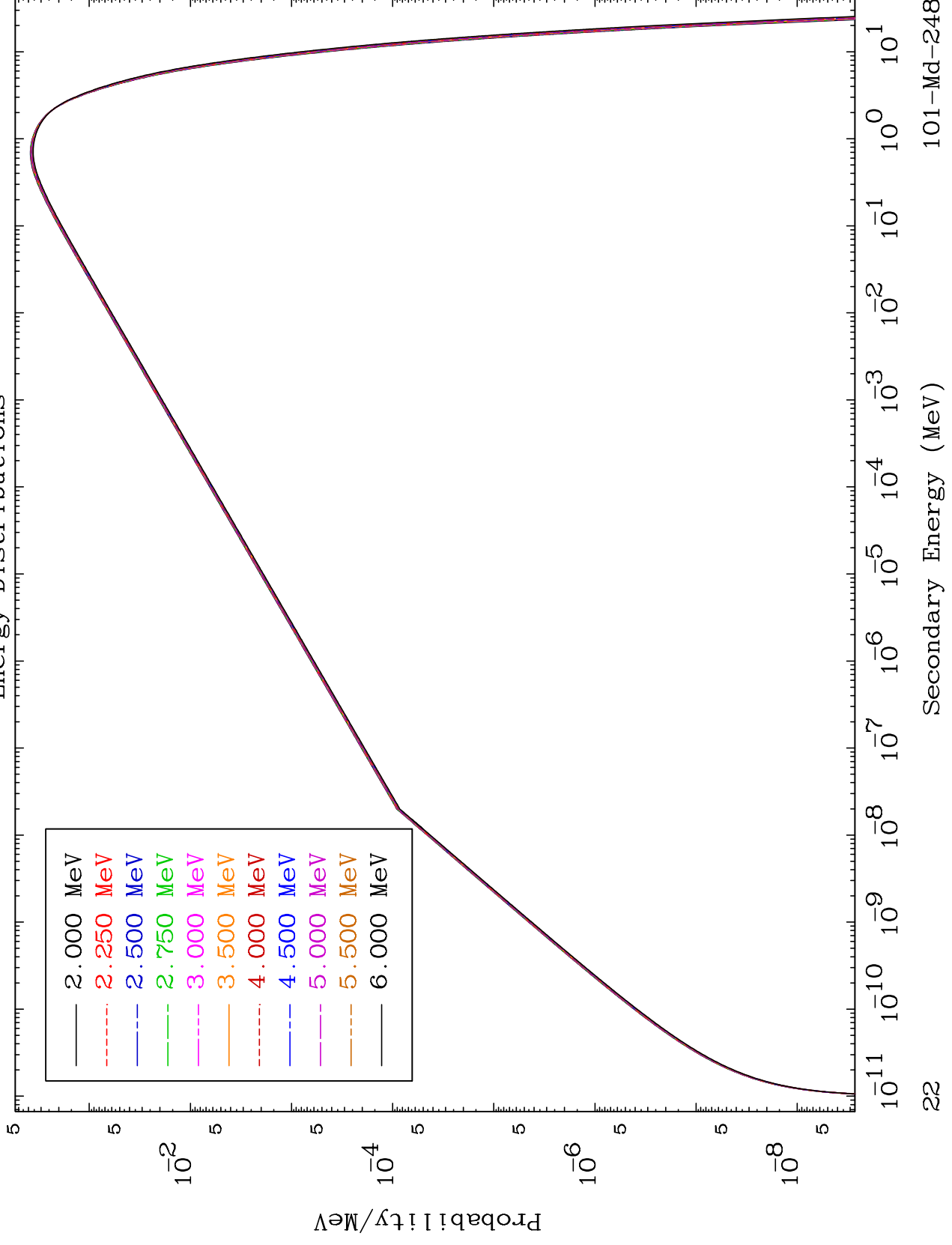
101-Md-248



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Fission Energy Distributions

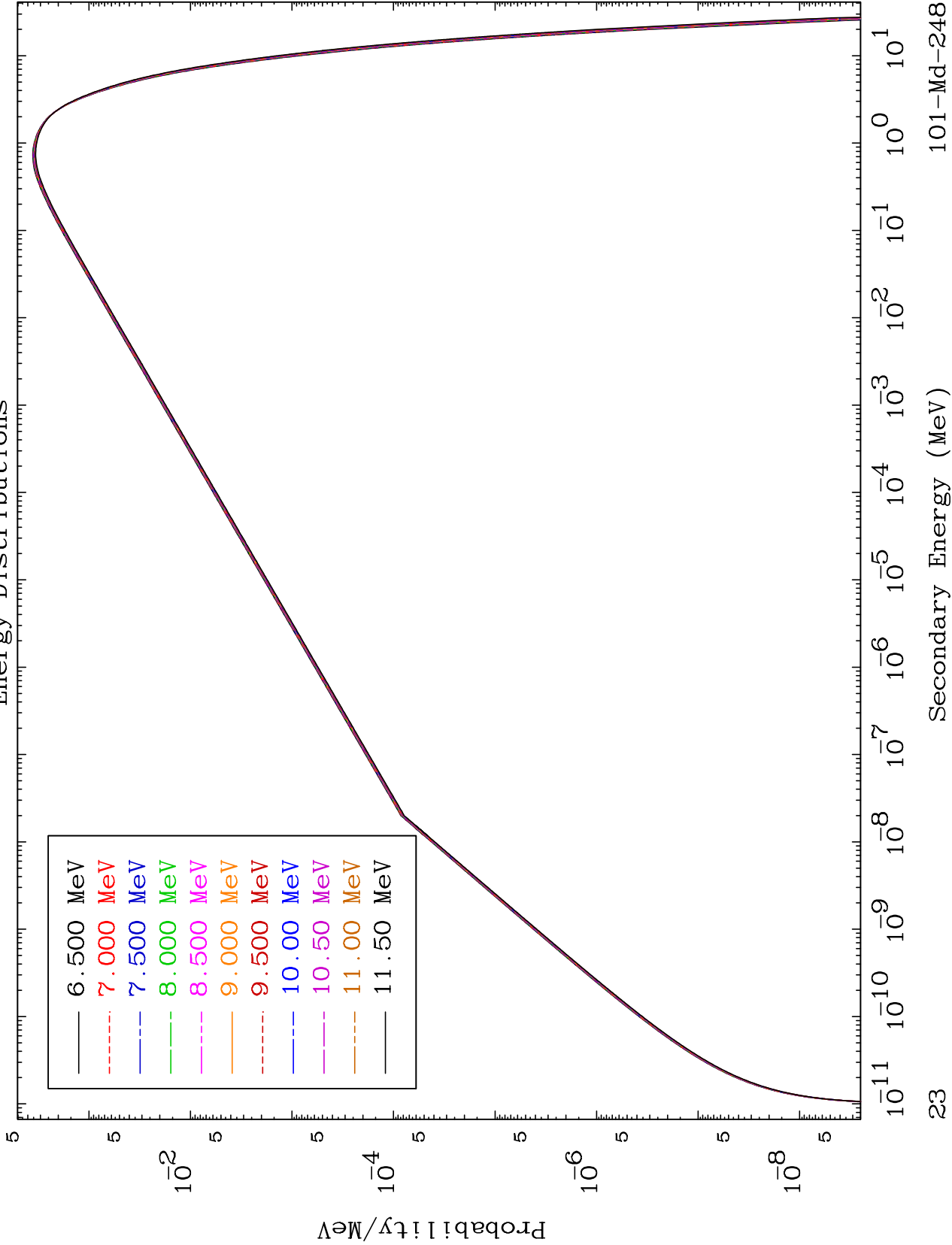
101-Md-248



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Fission
Energy Distributions

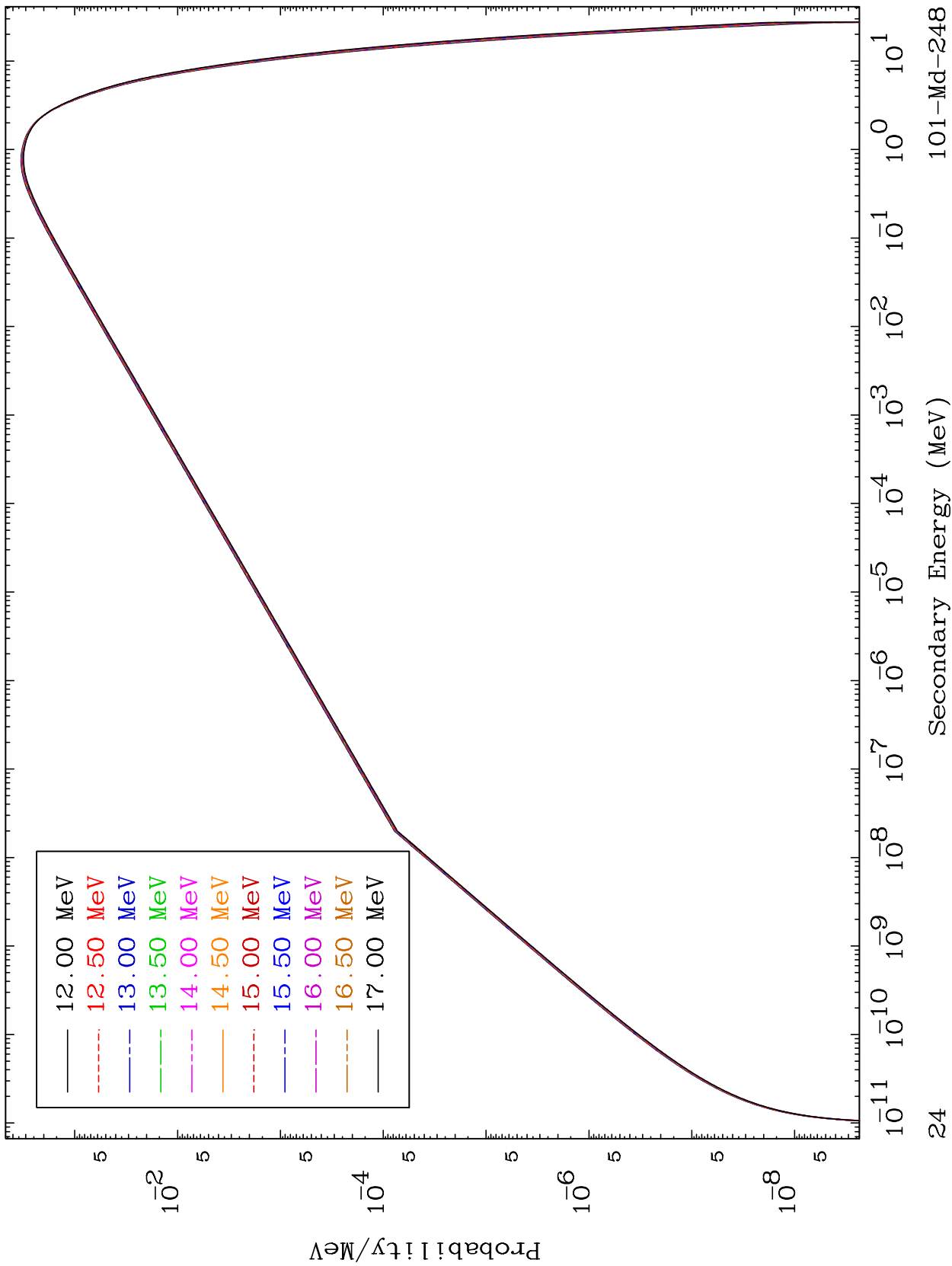
101-Md-248



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Fission Energy Distributions

101-Md-248



101-Md-248

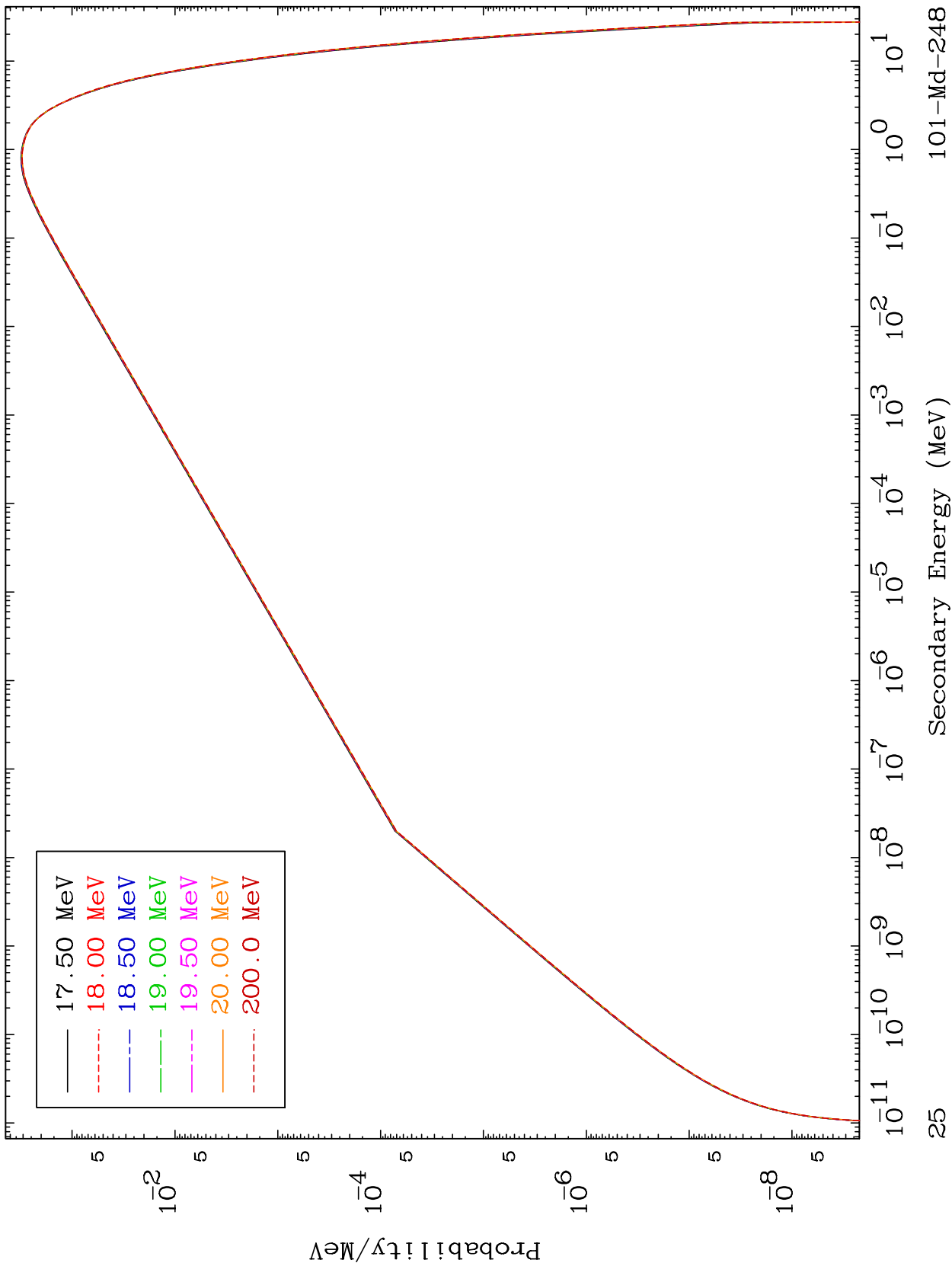
Secondary Energy (MeV)

24

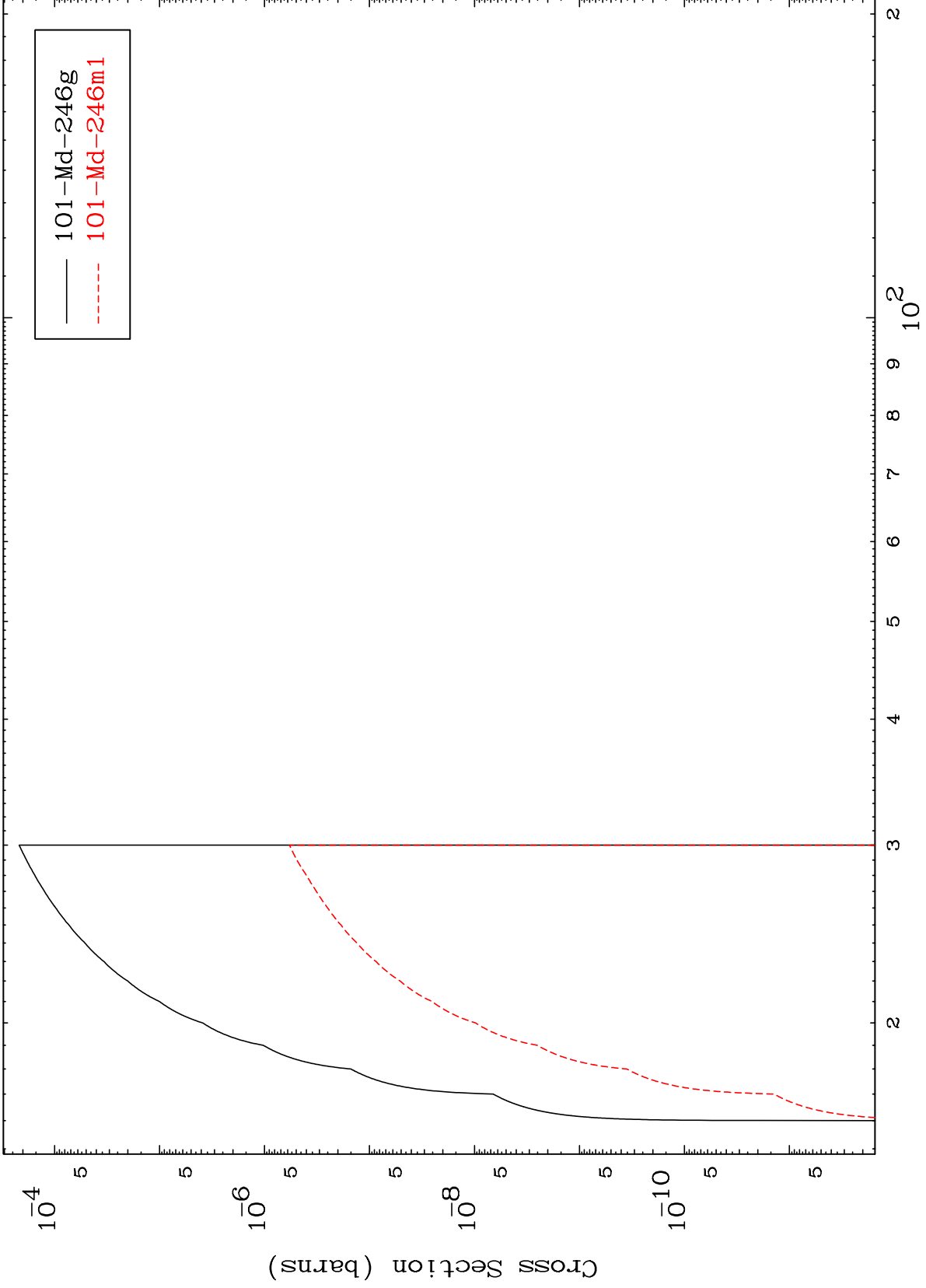
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Fission
Energy Distributions

101-Md-248



(n,3n)
Radionuclide Production Cross Section

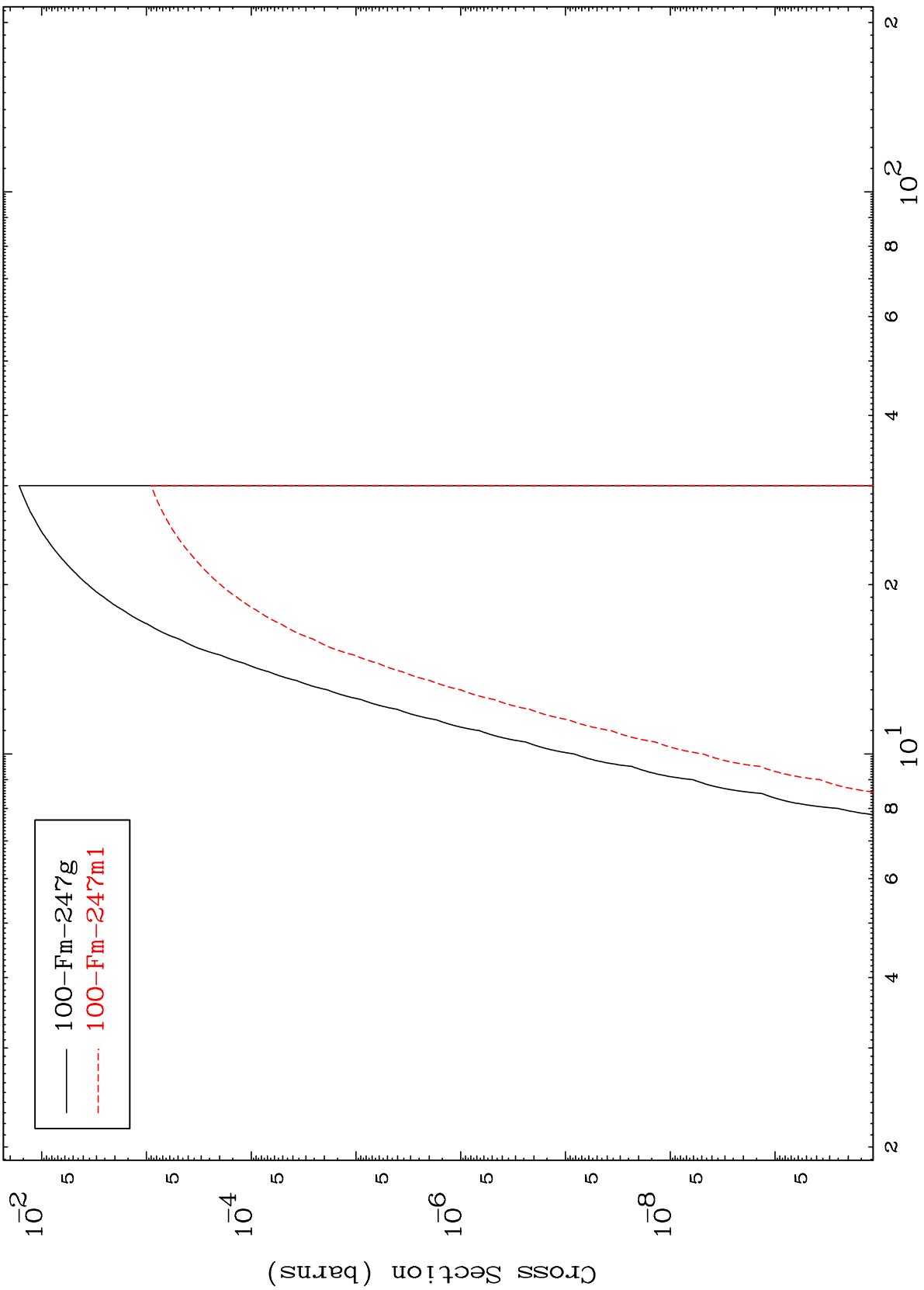


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(n,n') p

101-Md-248

Radionuclide Production Cross Section



— 100-Fm-247g
- - - 100-Fm-247m1

27

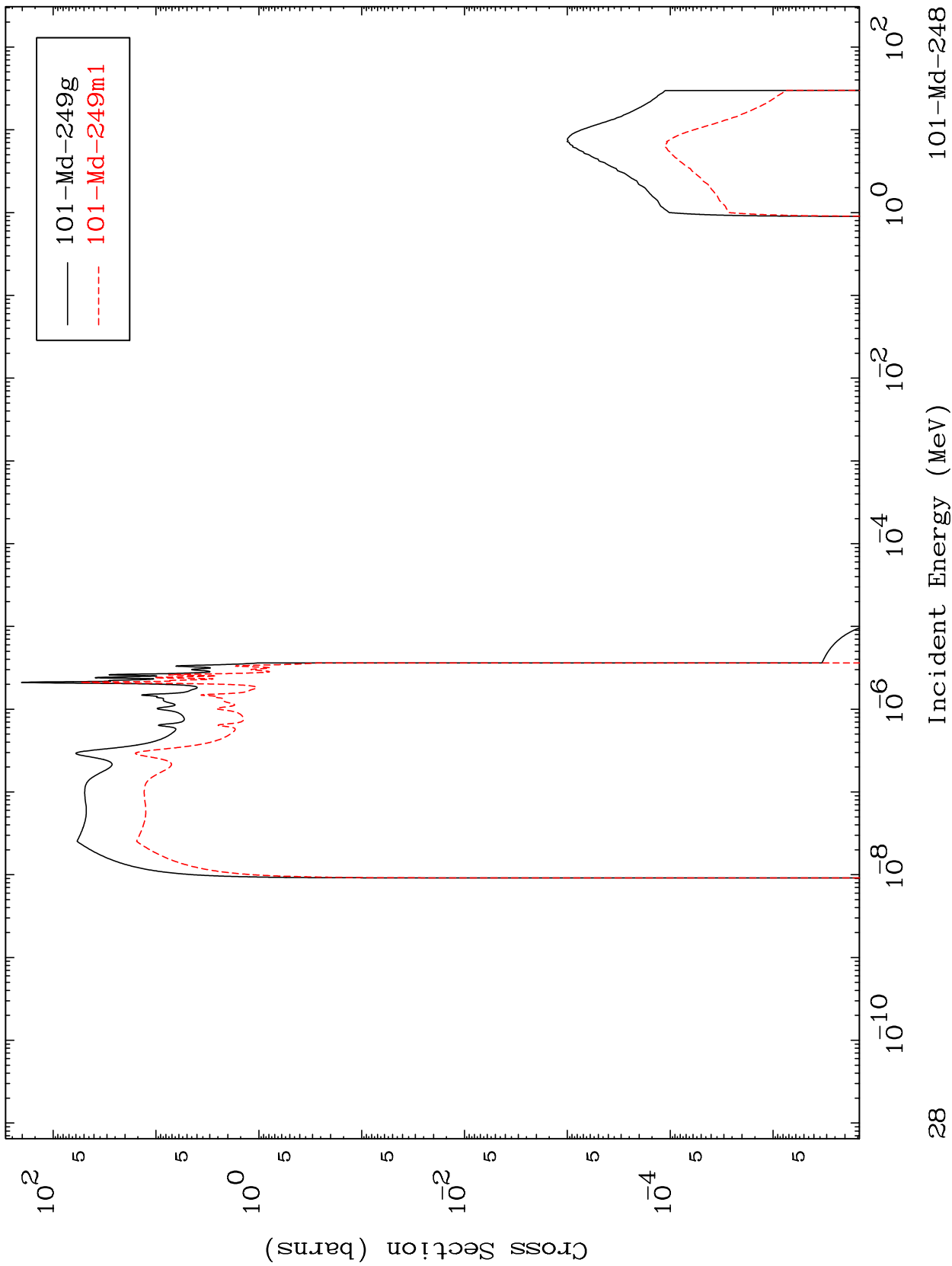
Incident Energy (MeV)

101-Md-248

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101-Md-248

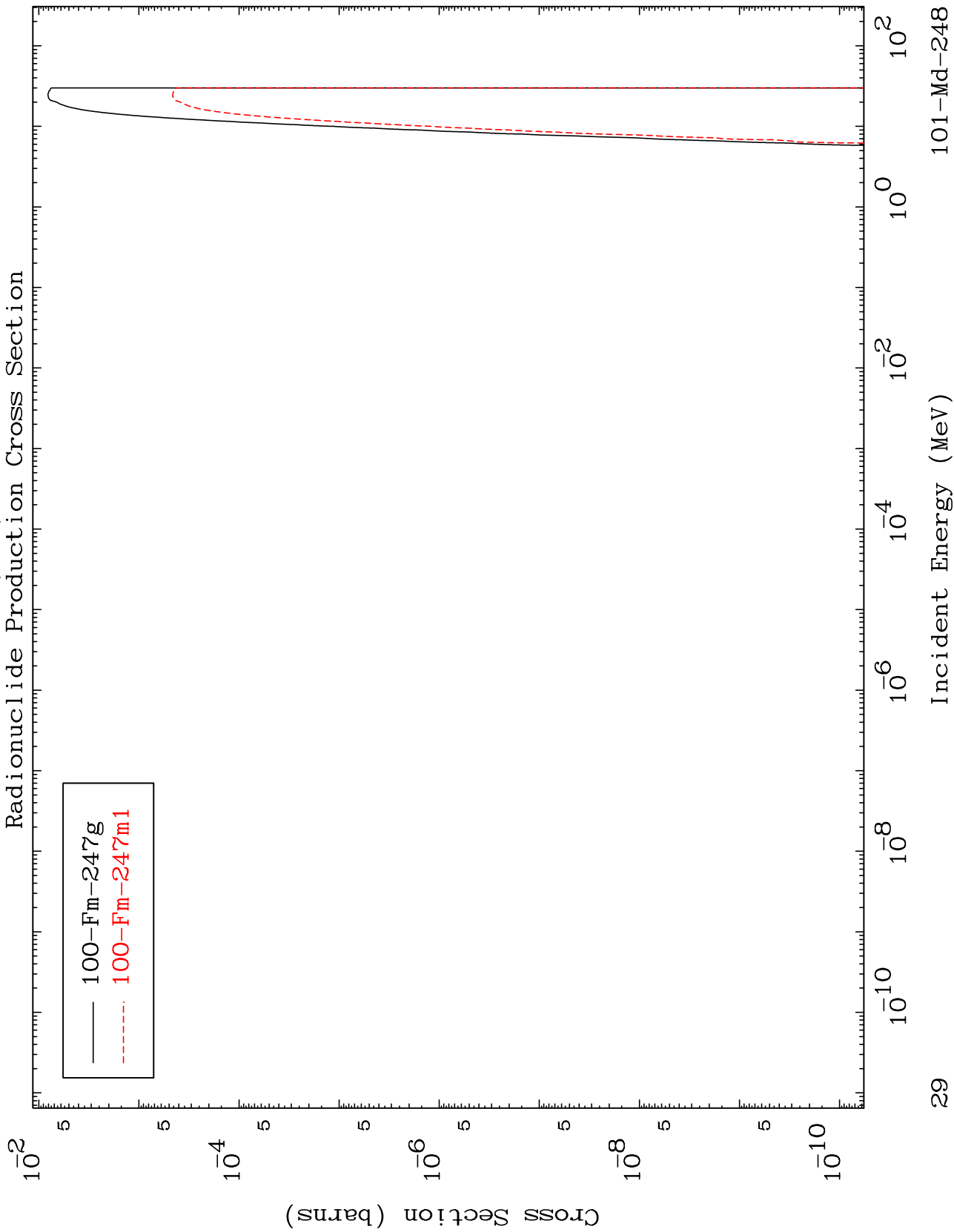
(n, γ)
Radionuclide Production Cross Section



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(n,d)

101-Md-248



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(n,2p)

101-Md-248

Radionuclide Production Cross Section

