

Program EVALPLOT  
(Version 2021-1)

by

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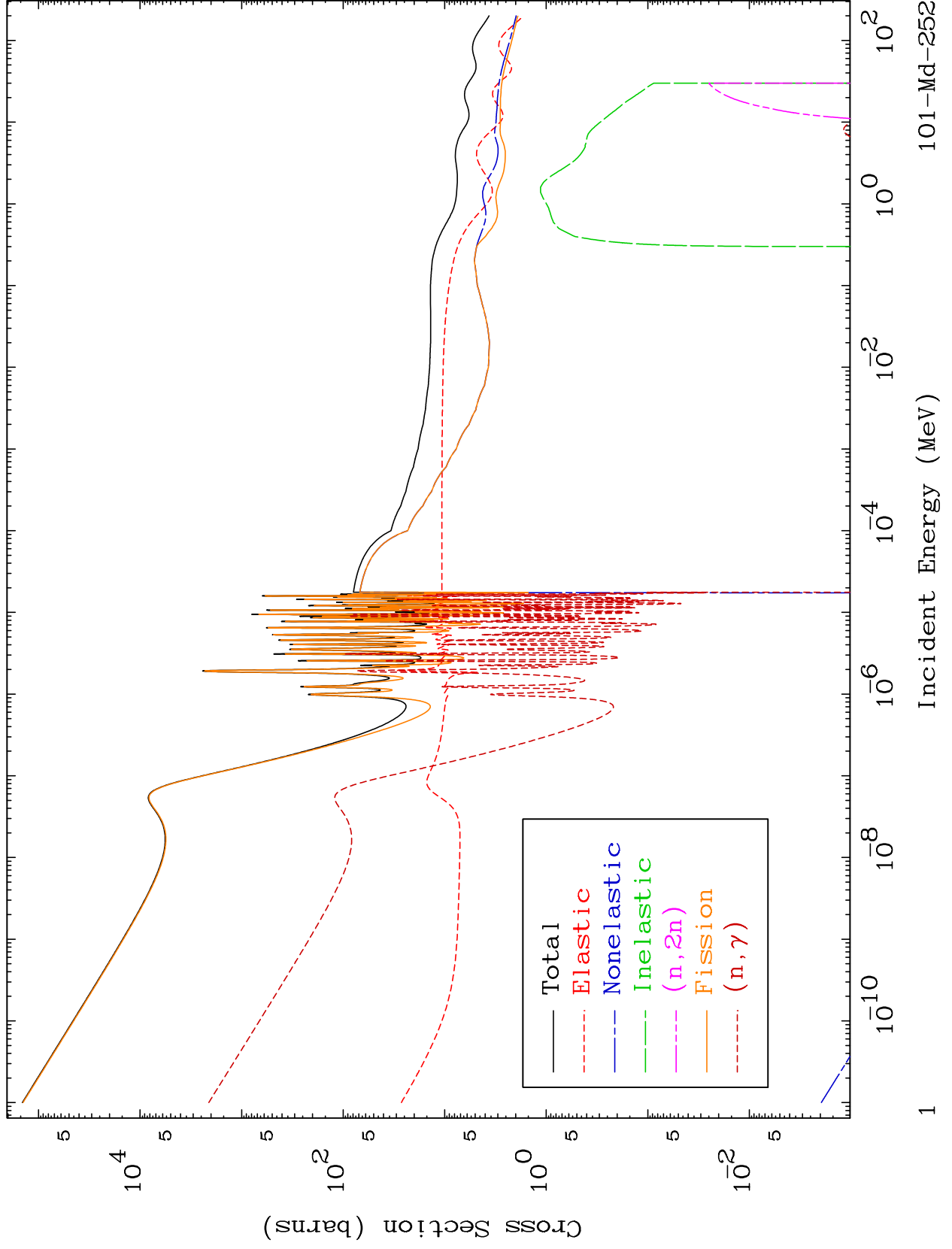
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9953

Neutron Major  
293 Kelvin Cross Sections

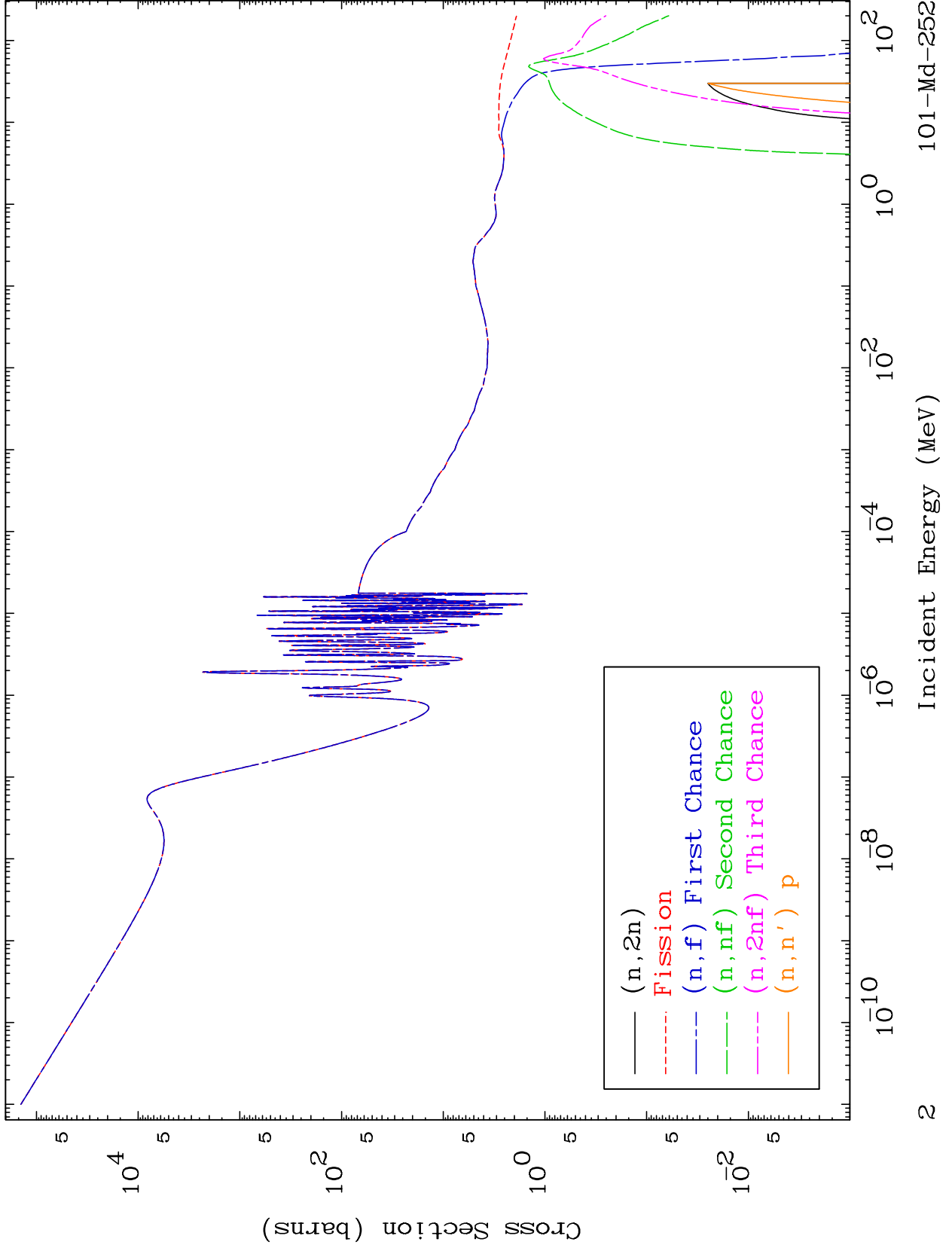
101-Md-252

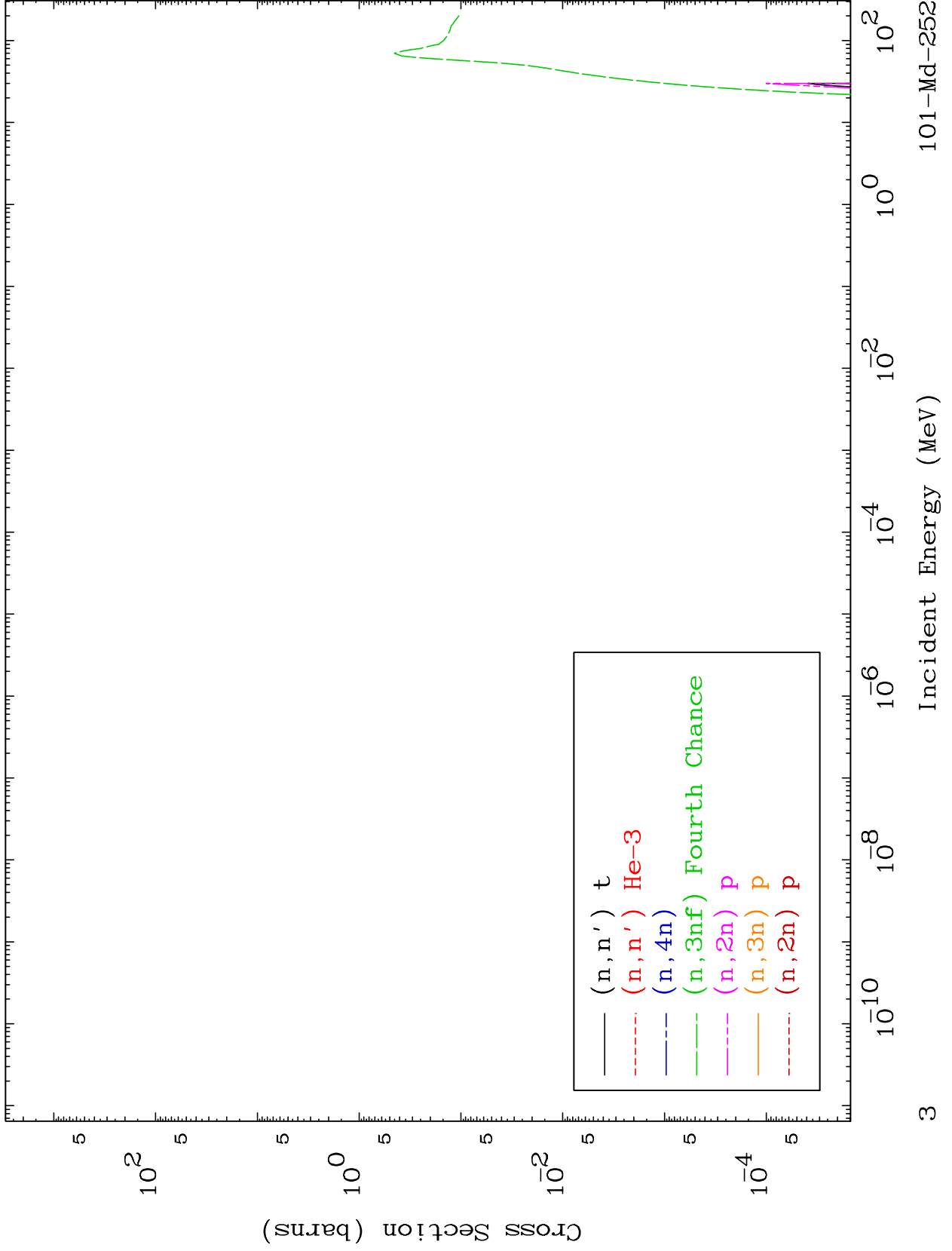


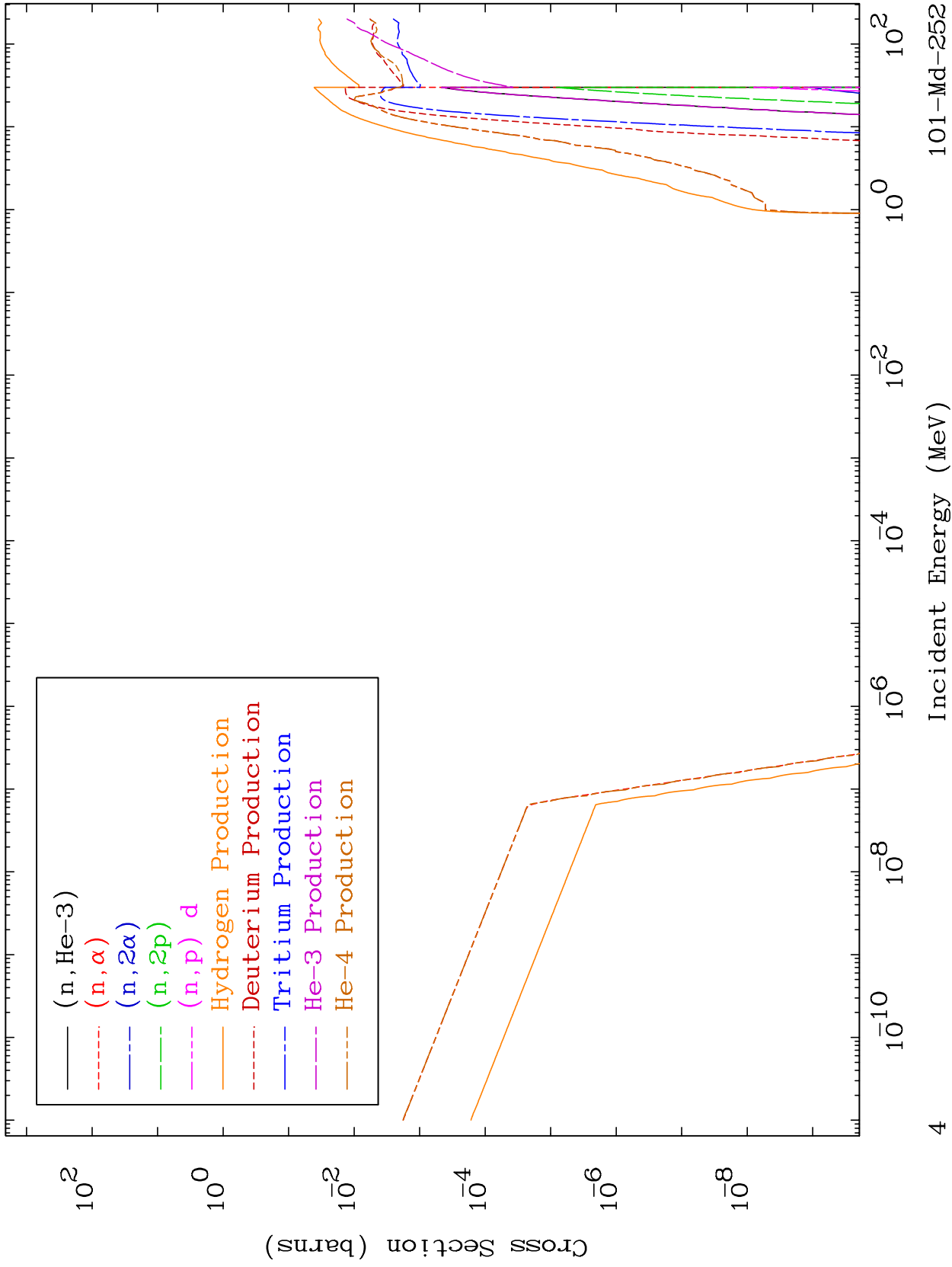
MAT 9953

Neutron Absorption  
293 Kelvin Cross Sections

101-Md-252



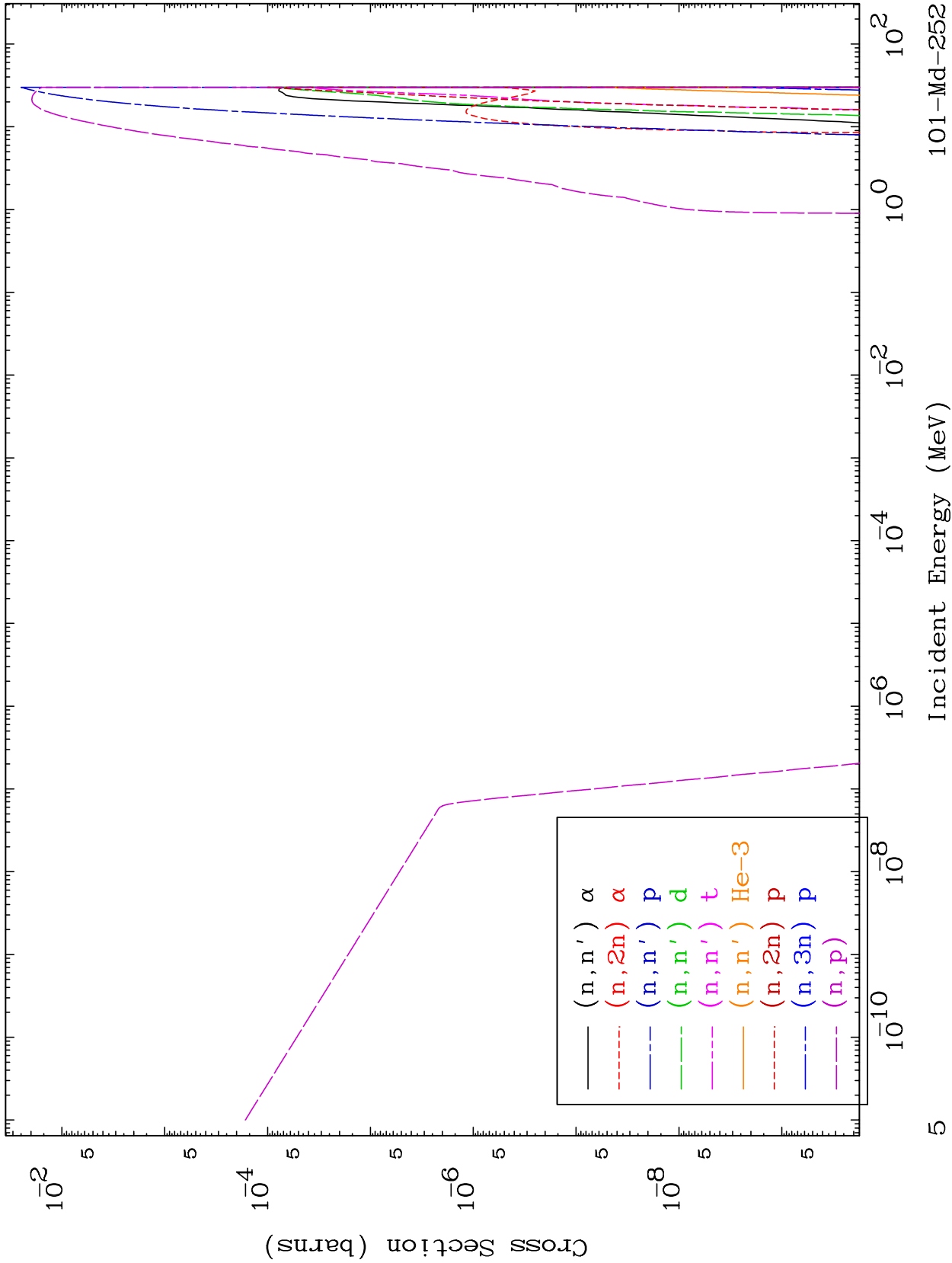




MAT 9953

Charged Particle  
293 Kelvin Cross Sections

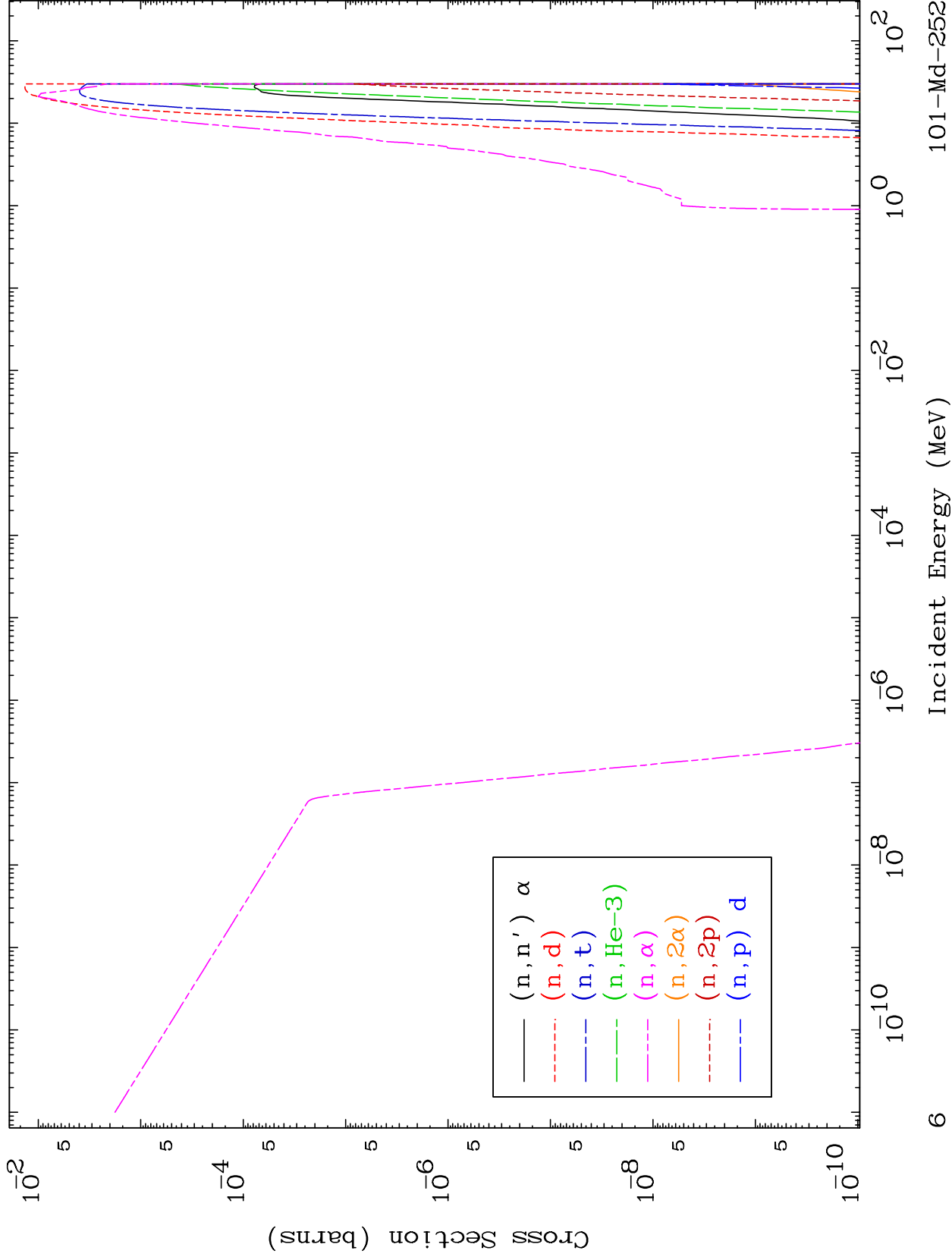
101-Md-252



MAT 9953

Charged Particle  
293 Kelvin Cross Sections

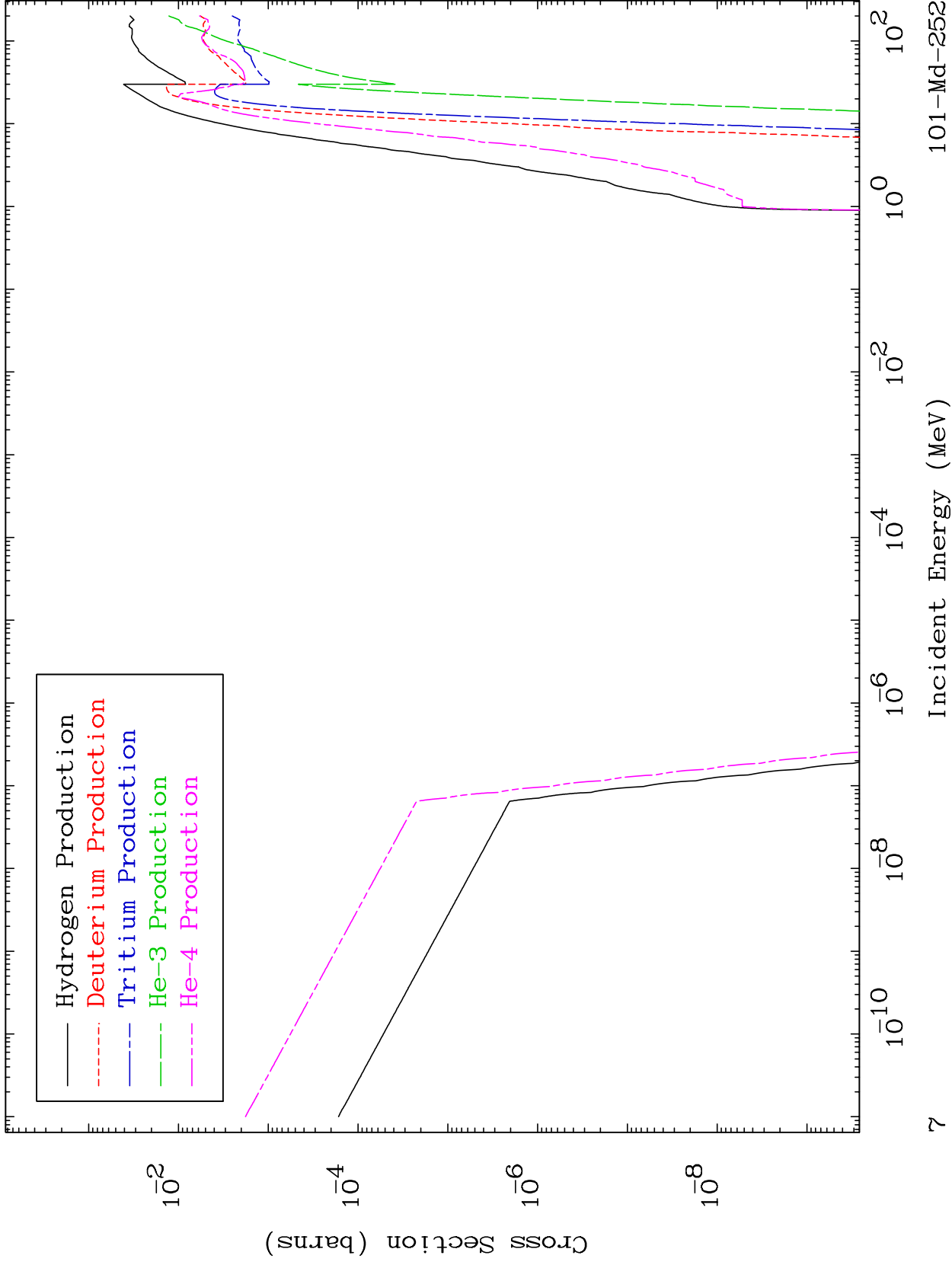
101-Md-252



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Particle Production  
293 Kelvin Cross Sections

101-Md-252

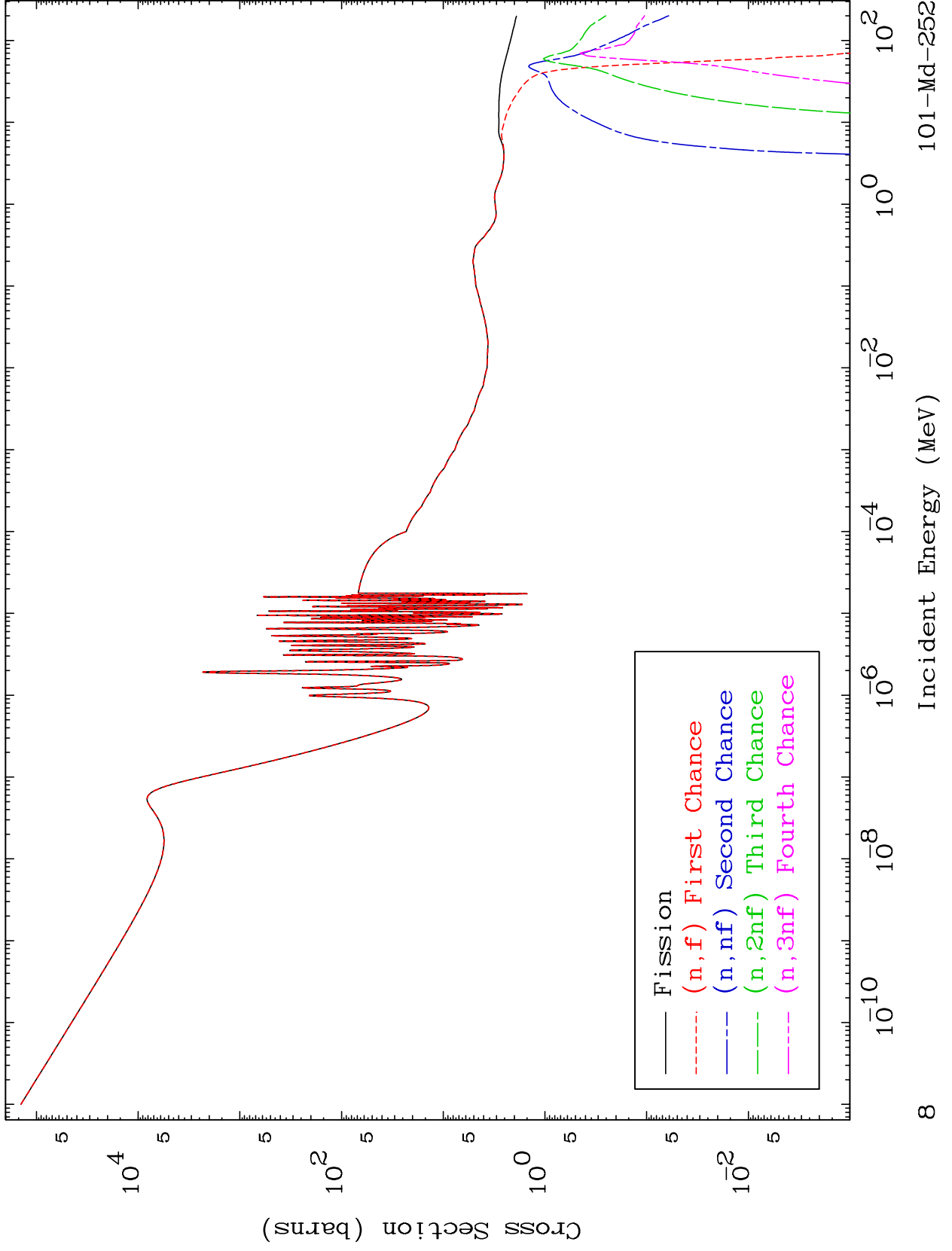




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Fission  
293 Kelvin Cross Sections

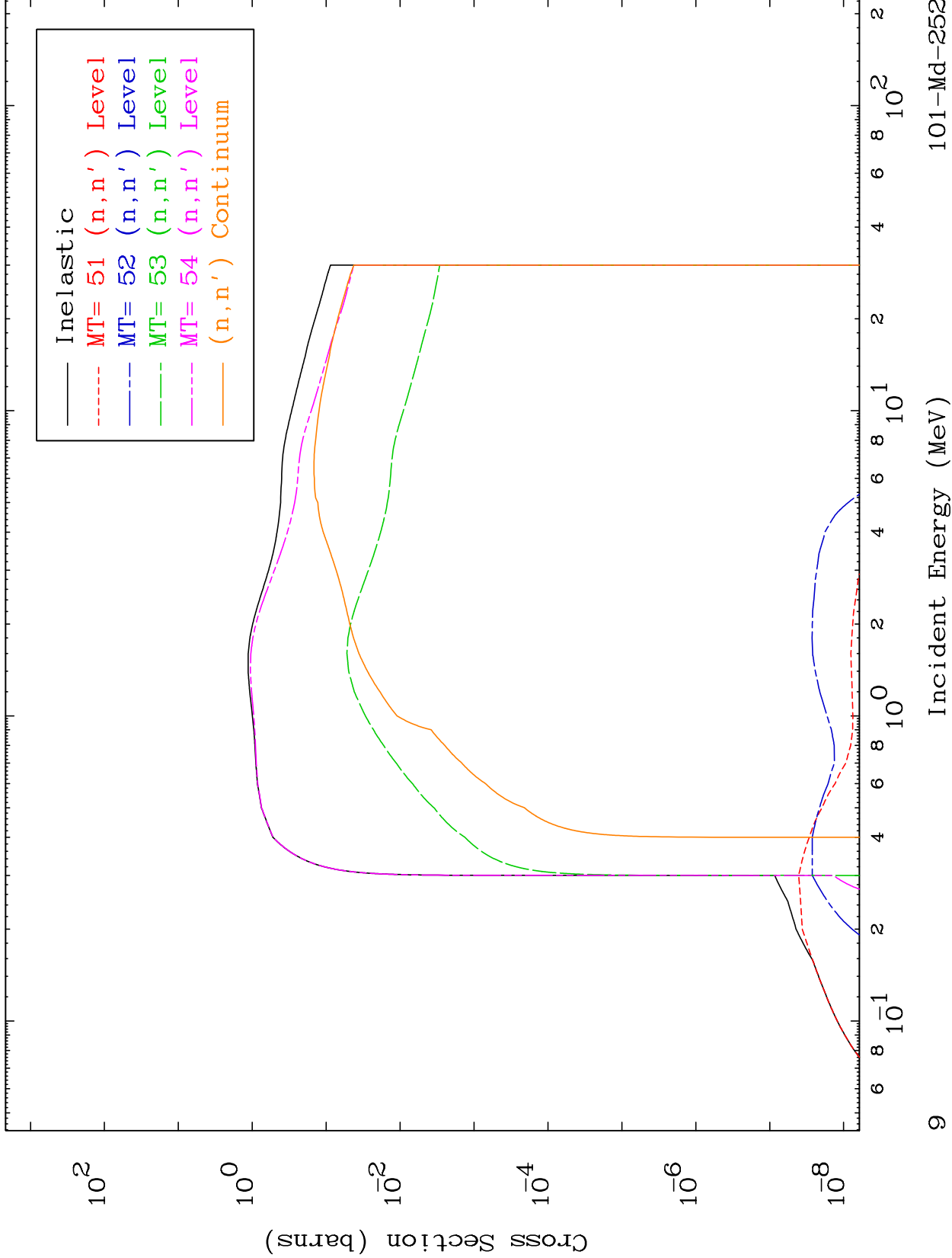
101-Md-252



MAT 9953

(n,n') Levels  
293 Kelvin Cross Sections

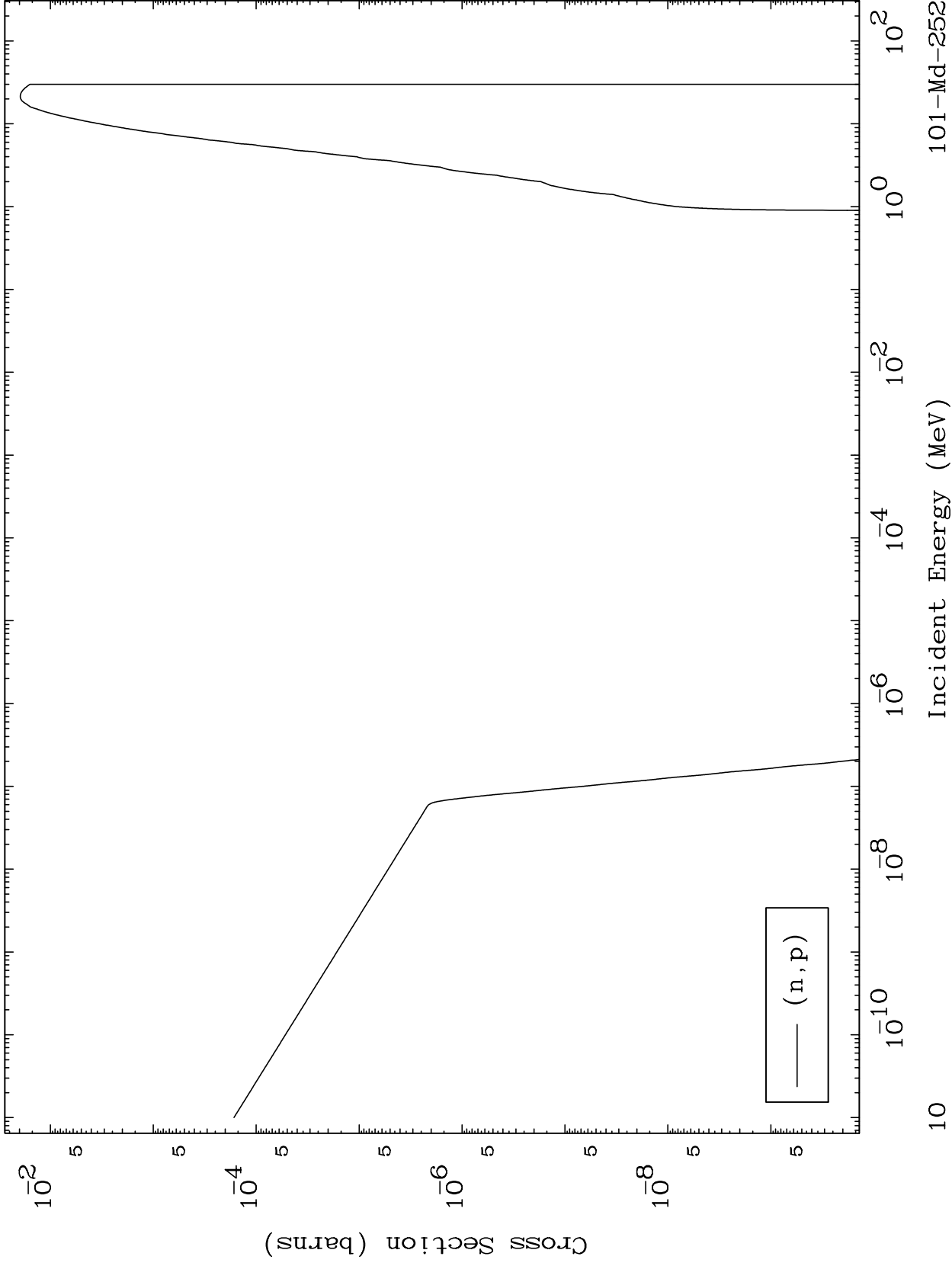
101-Md-252



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(n,p) Levels  
293 Kelvin Cross Sections

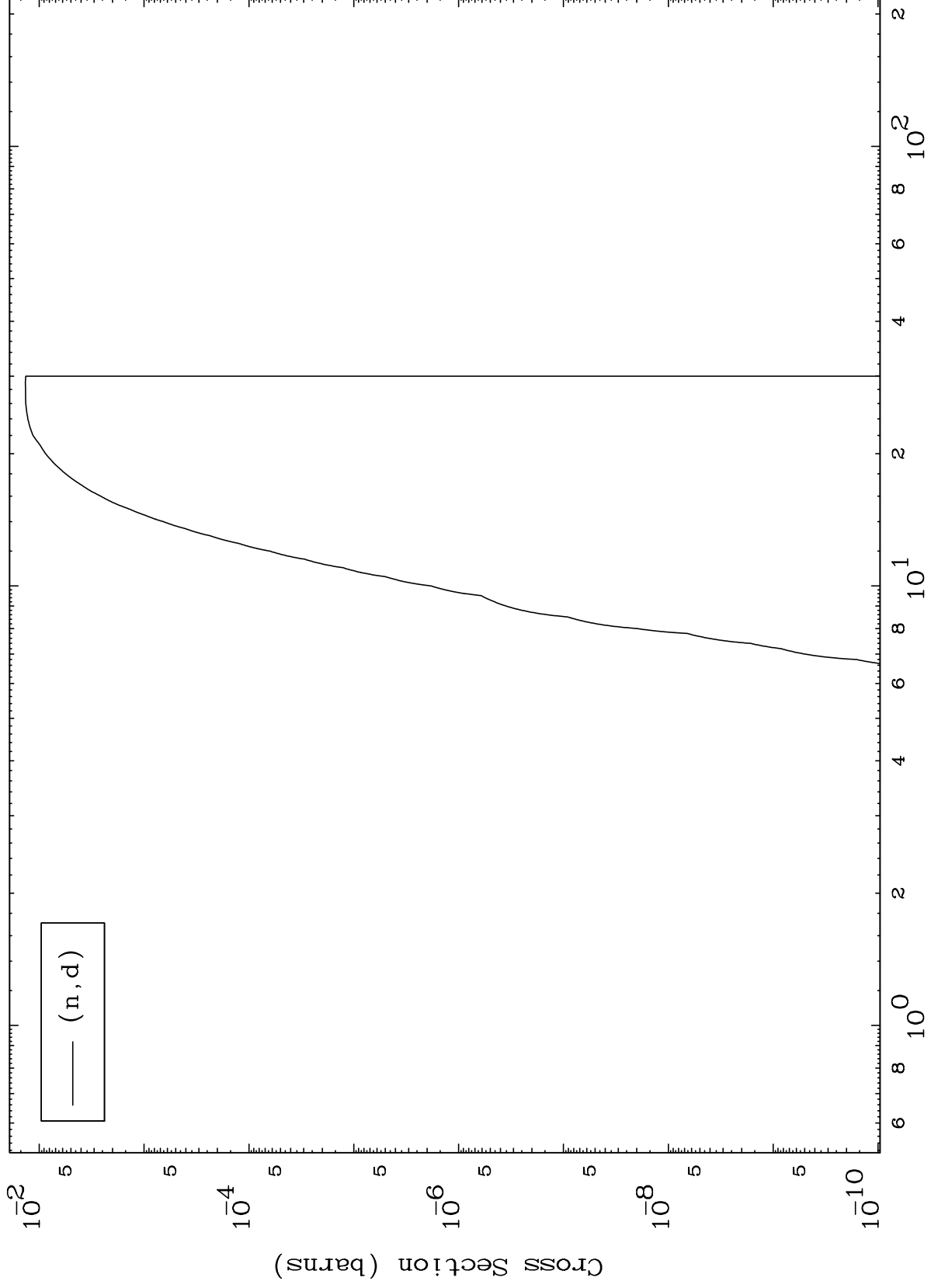
101-Md-252



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(n,d) Levels  
293 Kelvin Cross Sections

101-Md-252

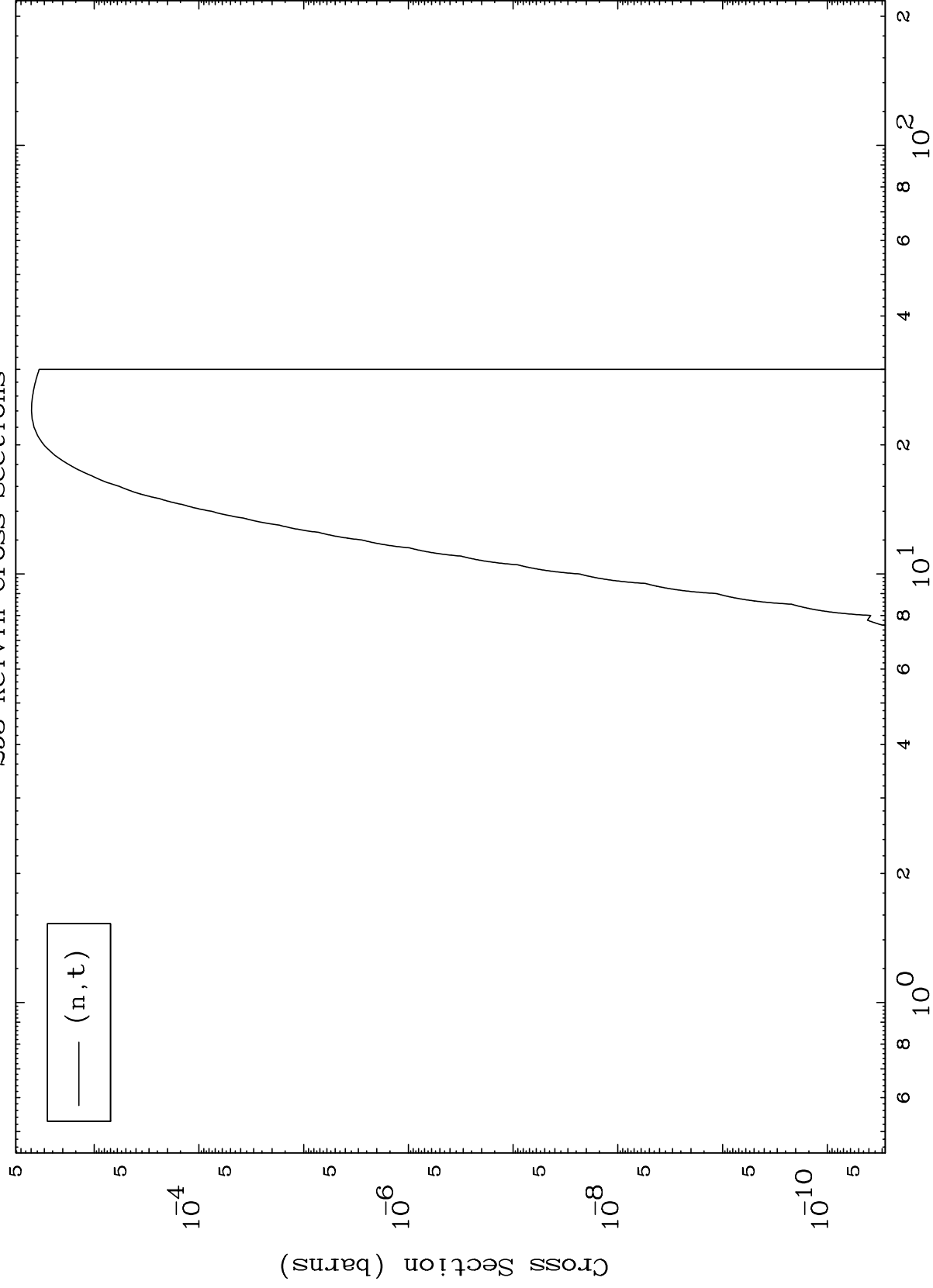


(n,d)

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(n,t) Levels  
293 Kelvin Cross Sections

101-Md-252



12

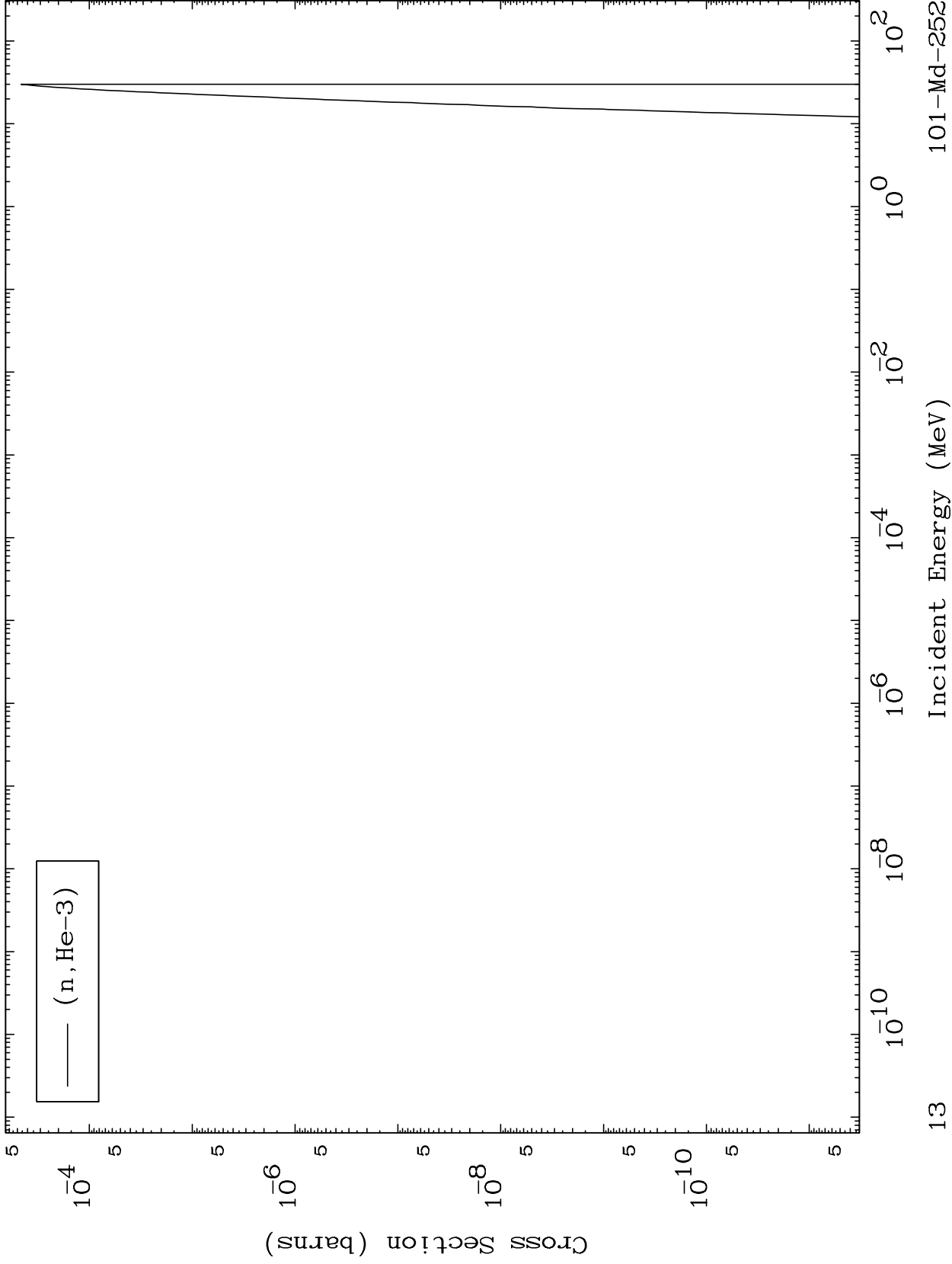
Incident Energy (MeV)

101-Md-252

MAT 9953

(n,He3) Levels  
293 Kelvin Cross Sections

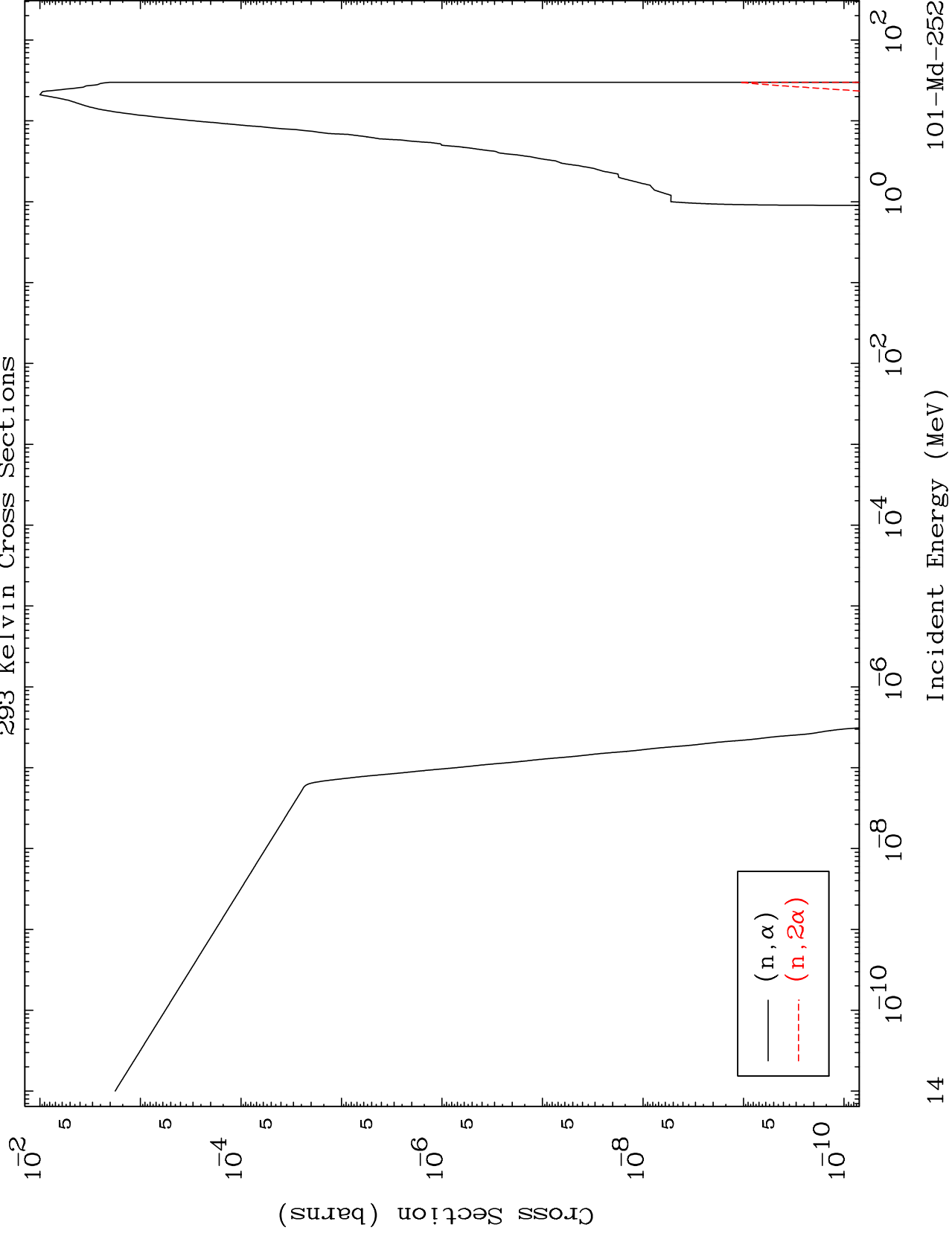
101-Md-252



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293 Kelvin Cross Sections  
(n,  $\alpha$ ) Levels

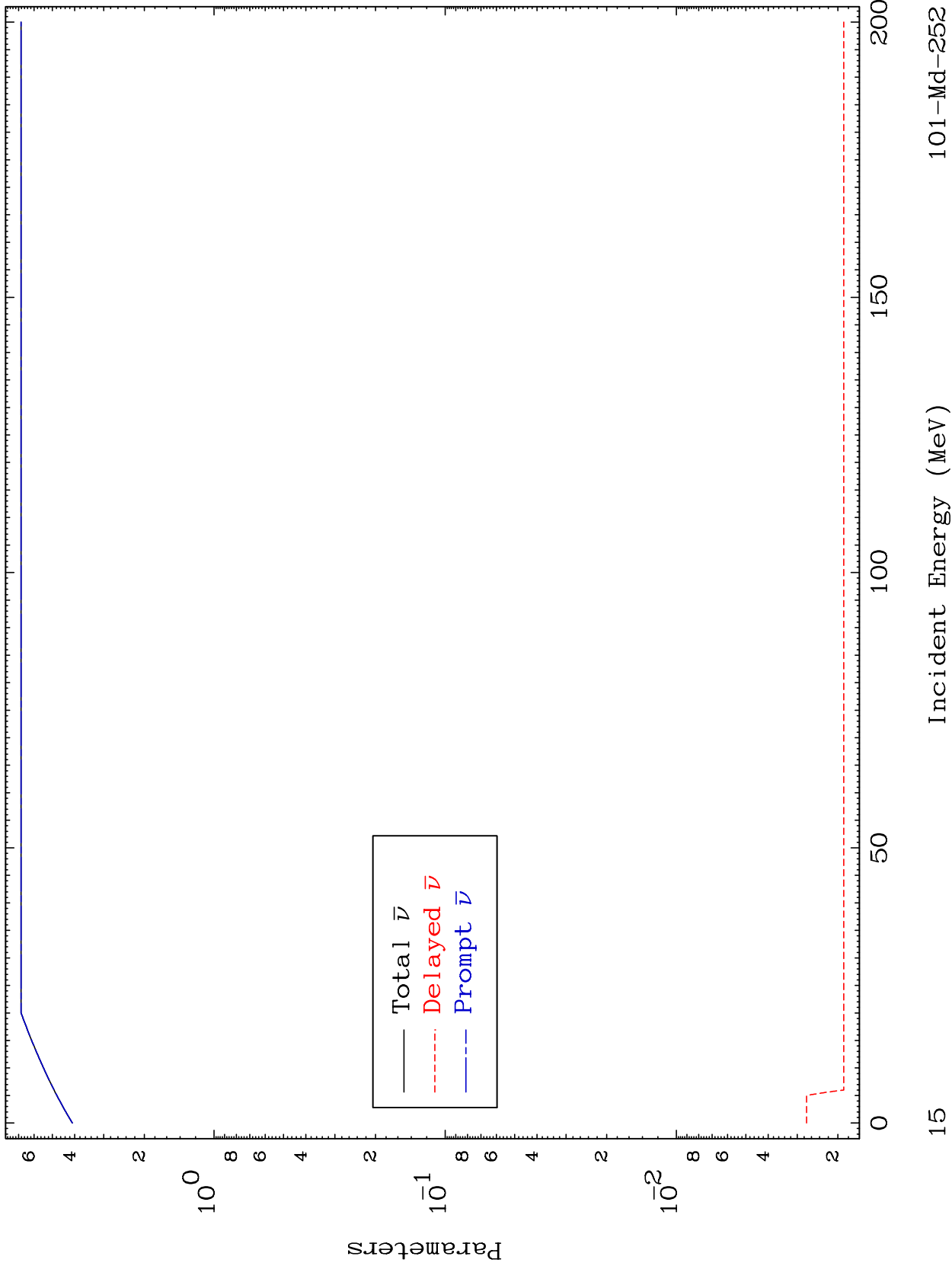
101-Md-252



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Energy Release  
Parameters

101-Md-252

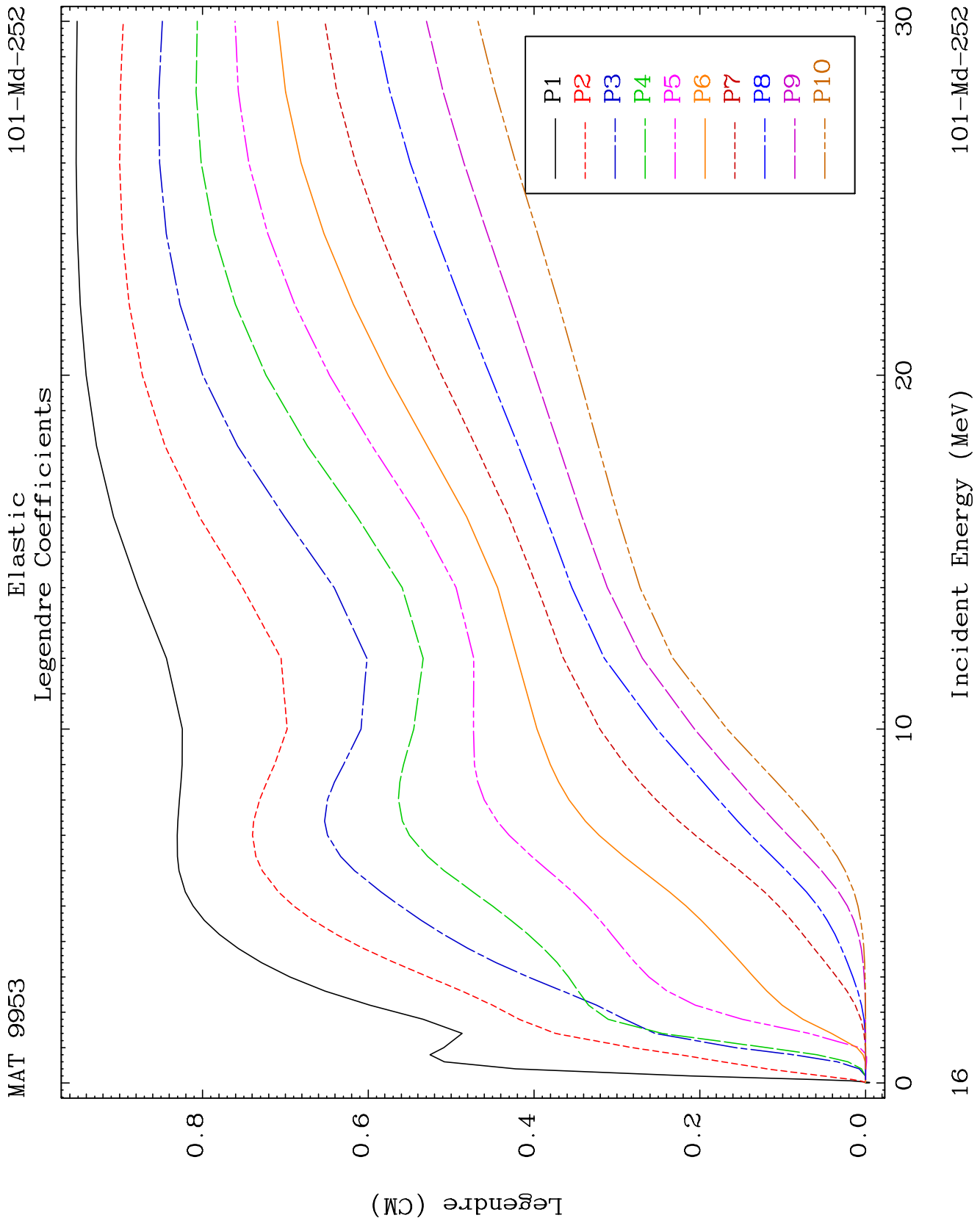


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Incident Energy (MeV)

101-Md-252

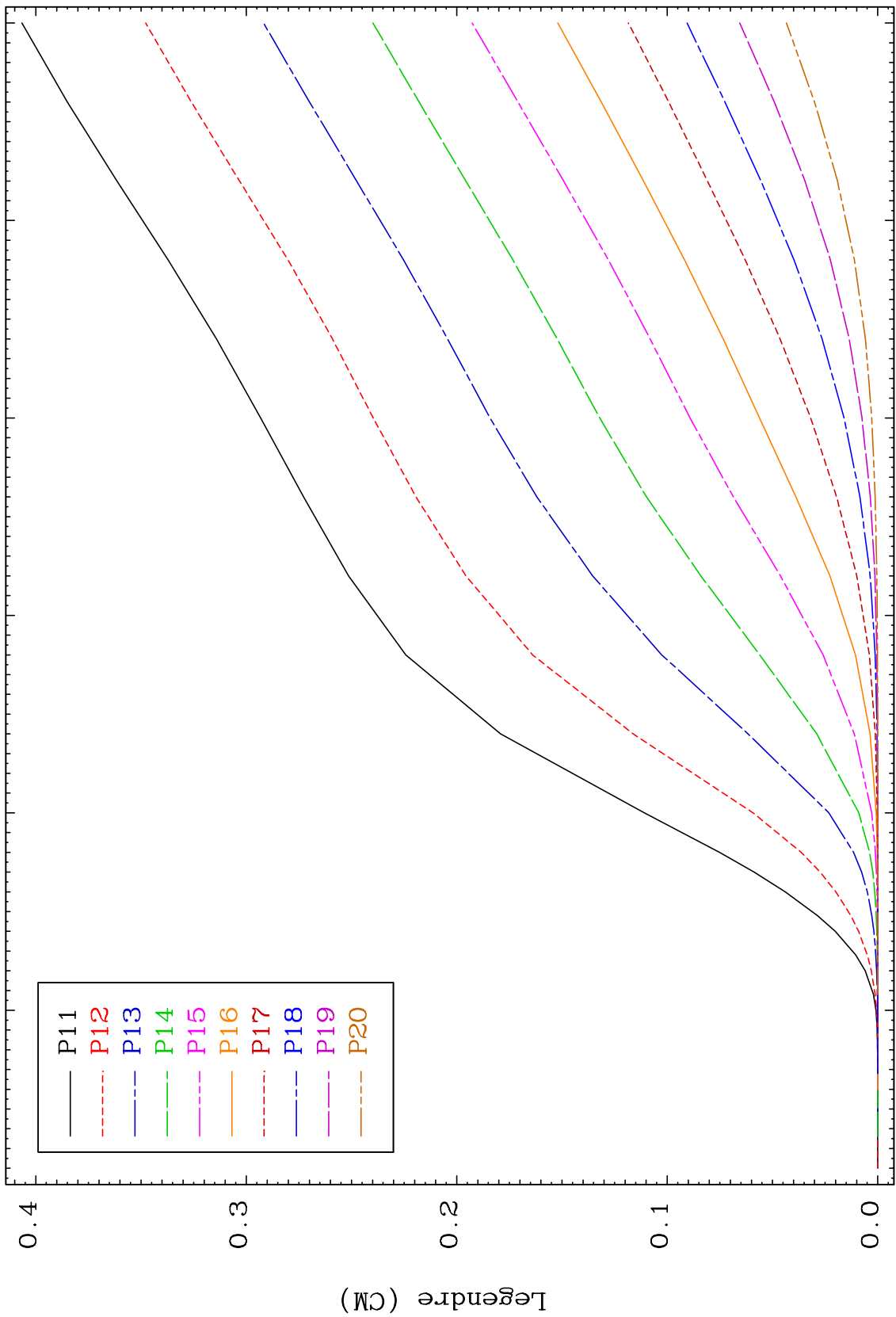




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Elastic Legendre Coefficients

101-Md-252



17

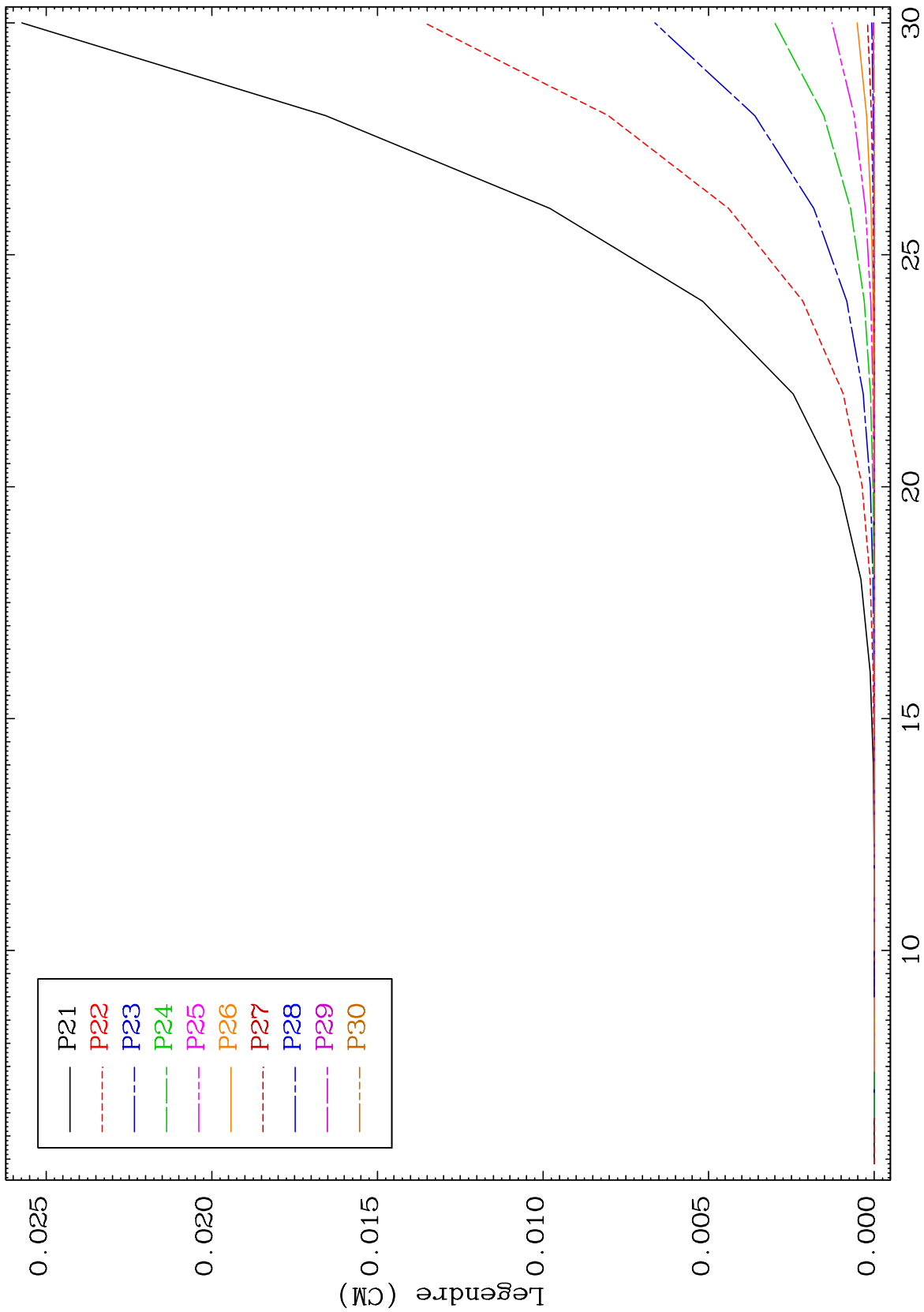
Incident Energy (MeV)

101-Md-252

MAT 9953

Elastic Legendre Coefficients

101-Md-252



18

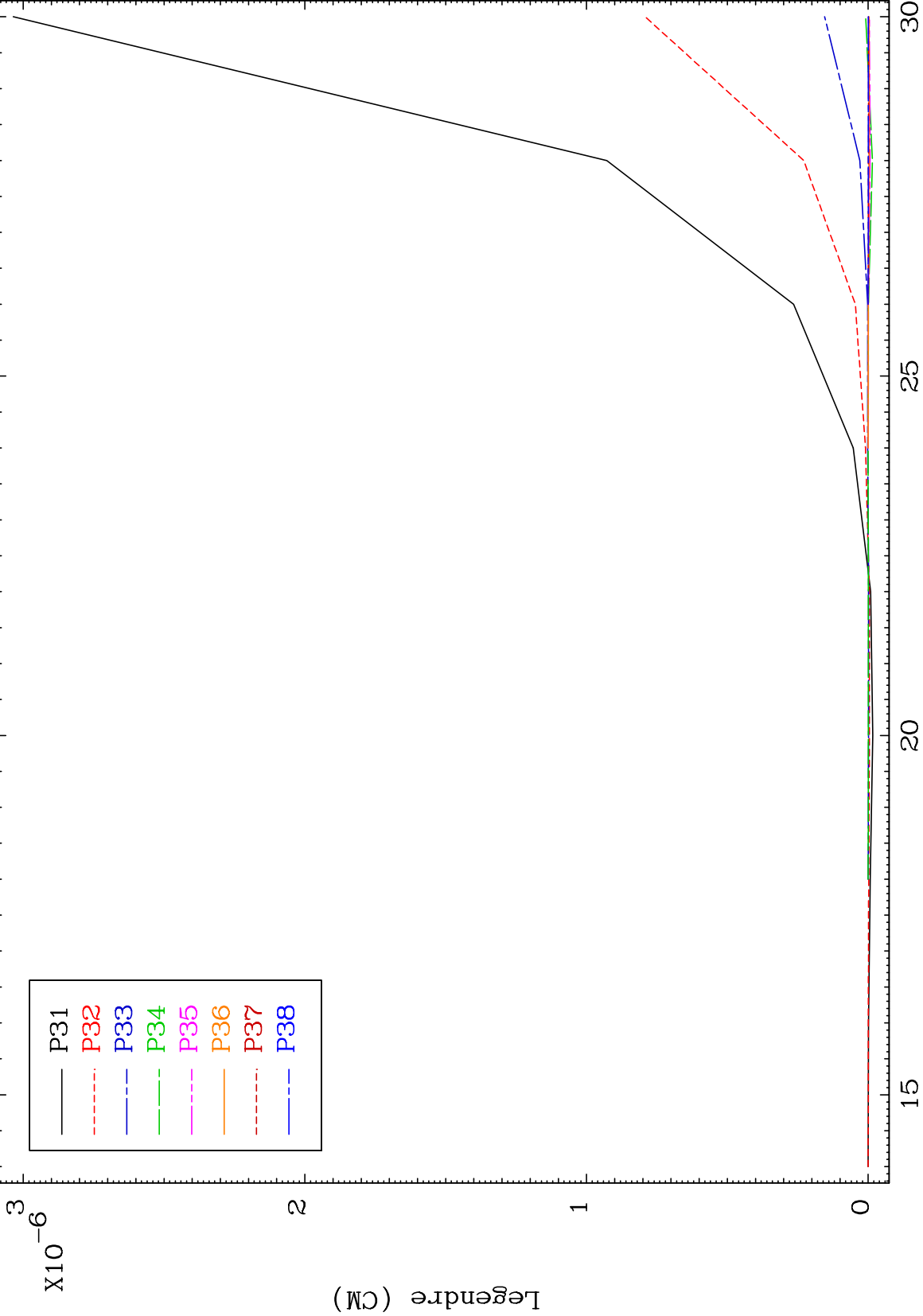
Incident Energy (MeV)

101-Md-252

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Elastic  
Legendre Coefficients

101-Md-252



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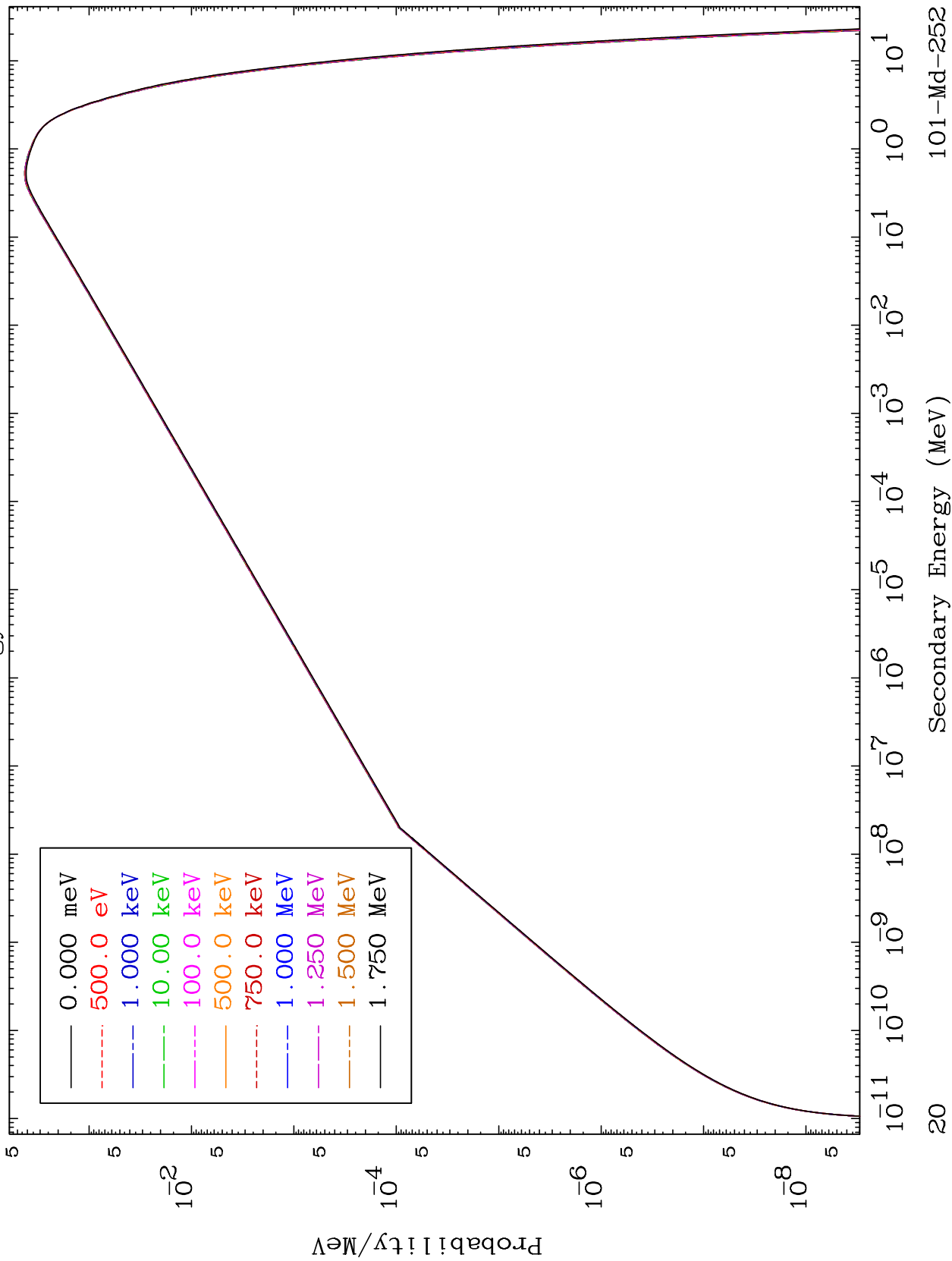
Incident Energy (MeV)

101-Md-252

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### Fission Energy Distributions

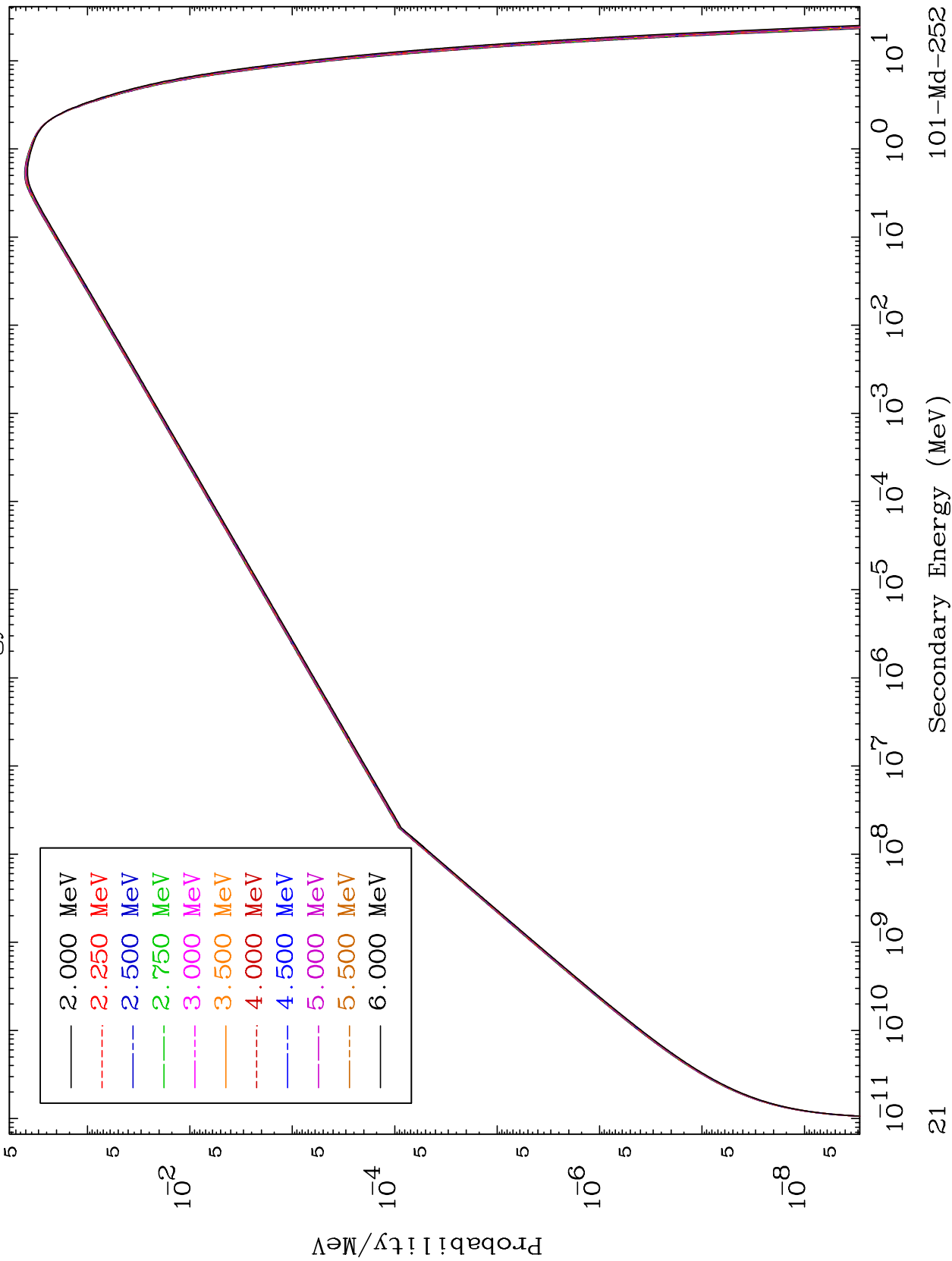
101-Md-252



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### Fission Energy Distributions

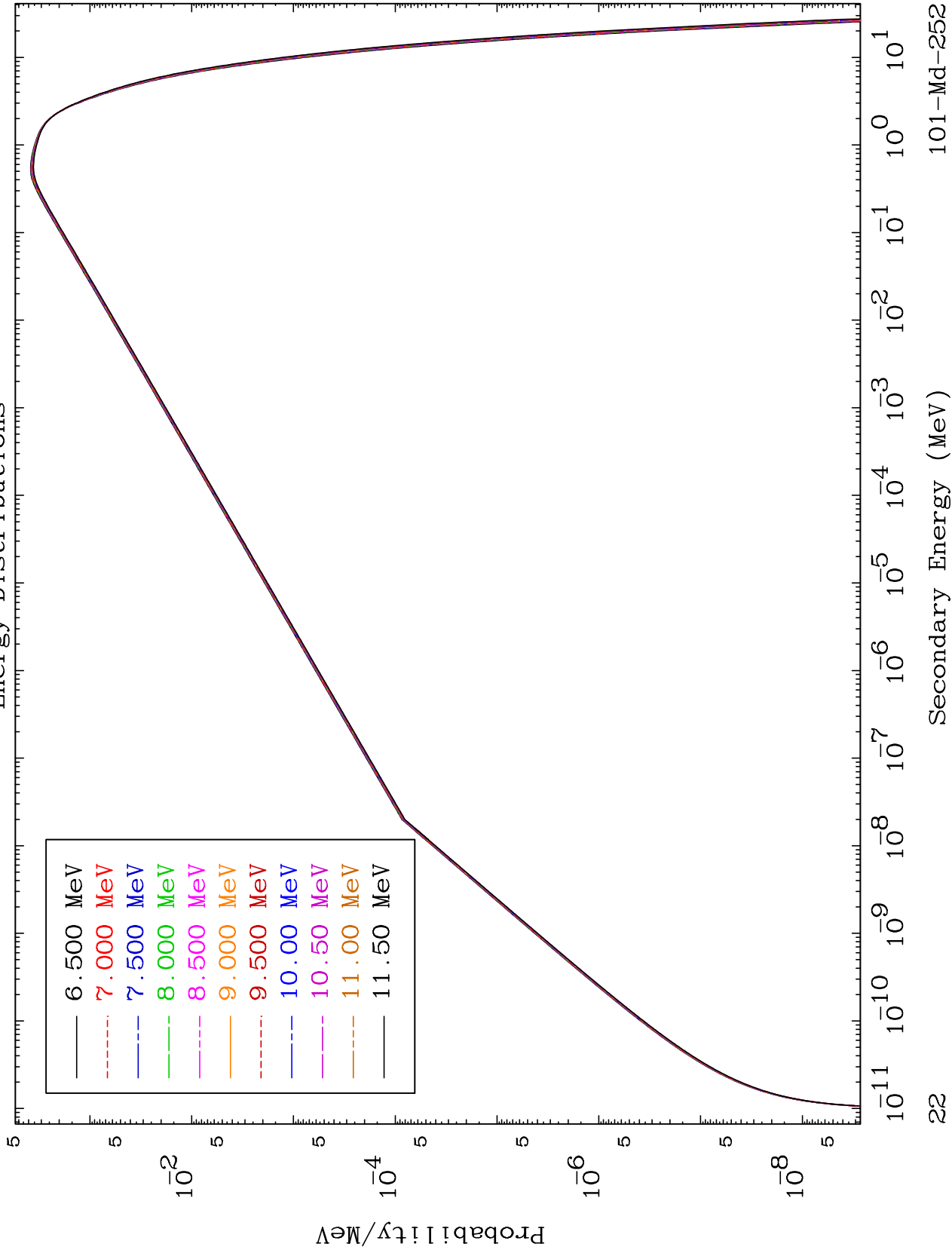
101-Md-252



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### Fission Energy Distributions

101-Md-252



101-Md-252

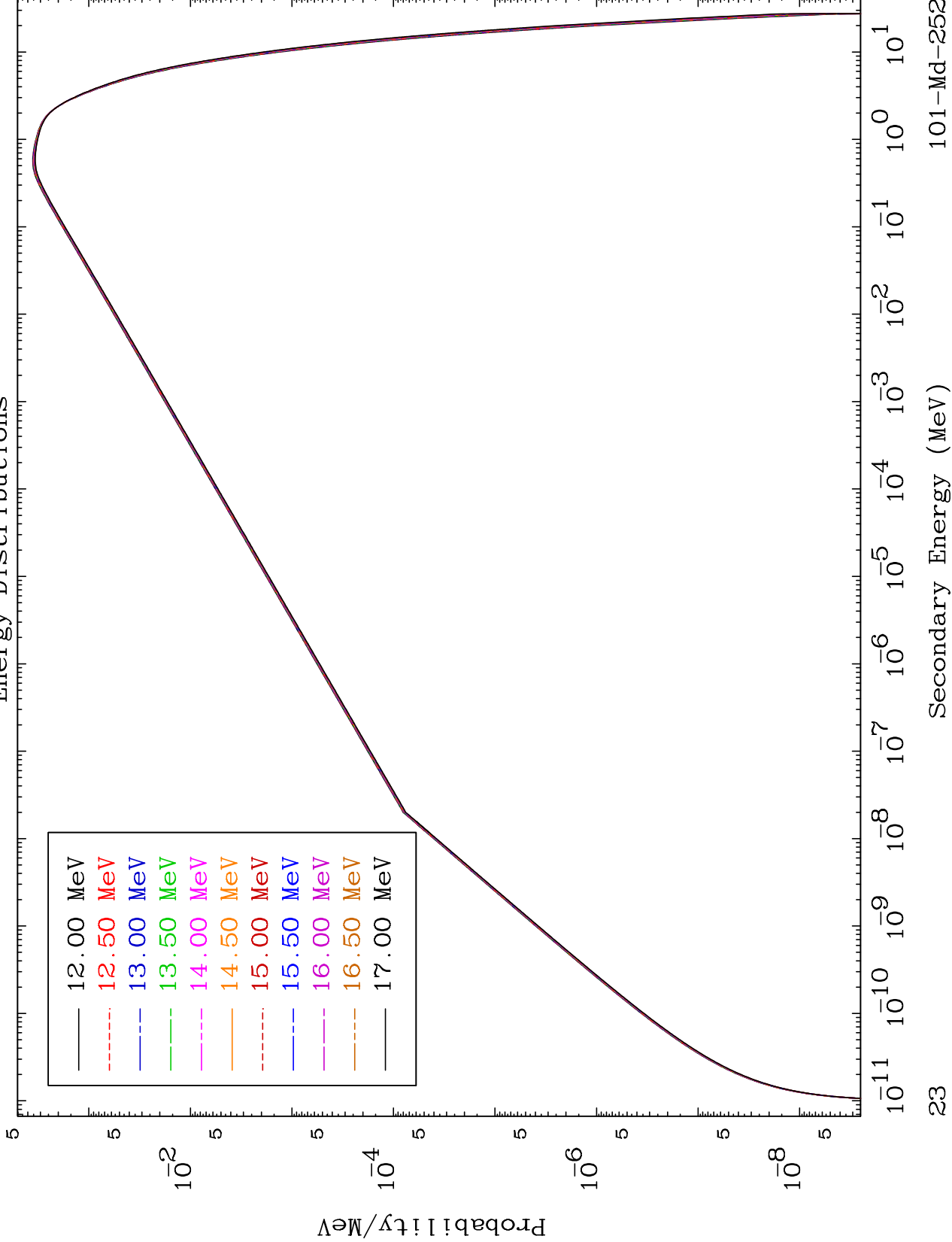
Secondary Energy (MeV)

22

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### Fission Energy Distributions

101-Md-252



101-Md-252

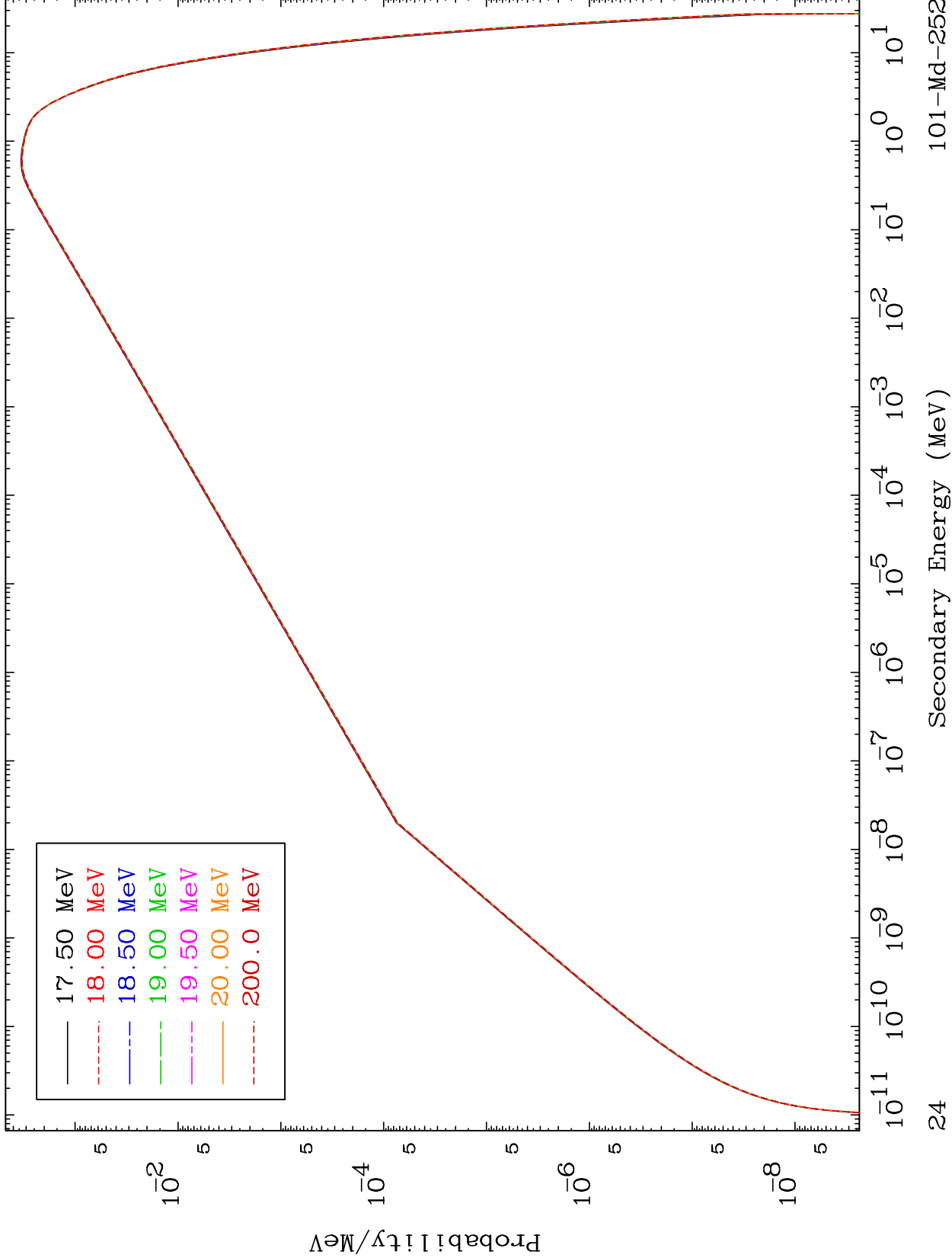
Secondary Energy (MeV)



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### Fission Energy Distributions

101-Md-252

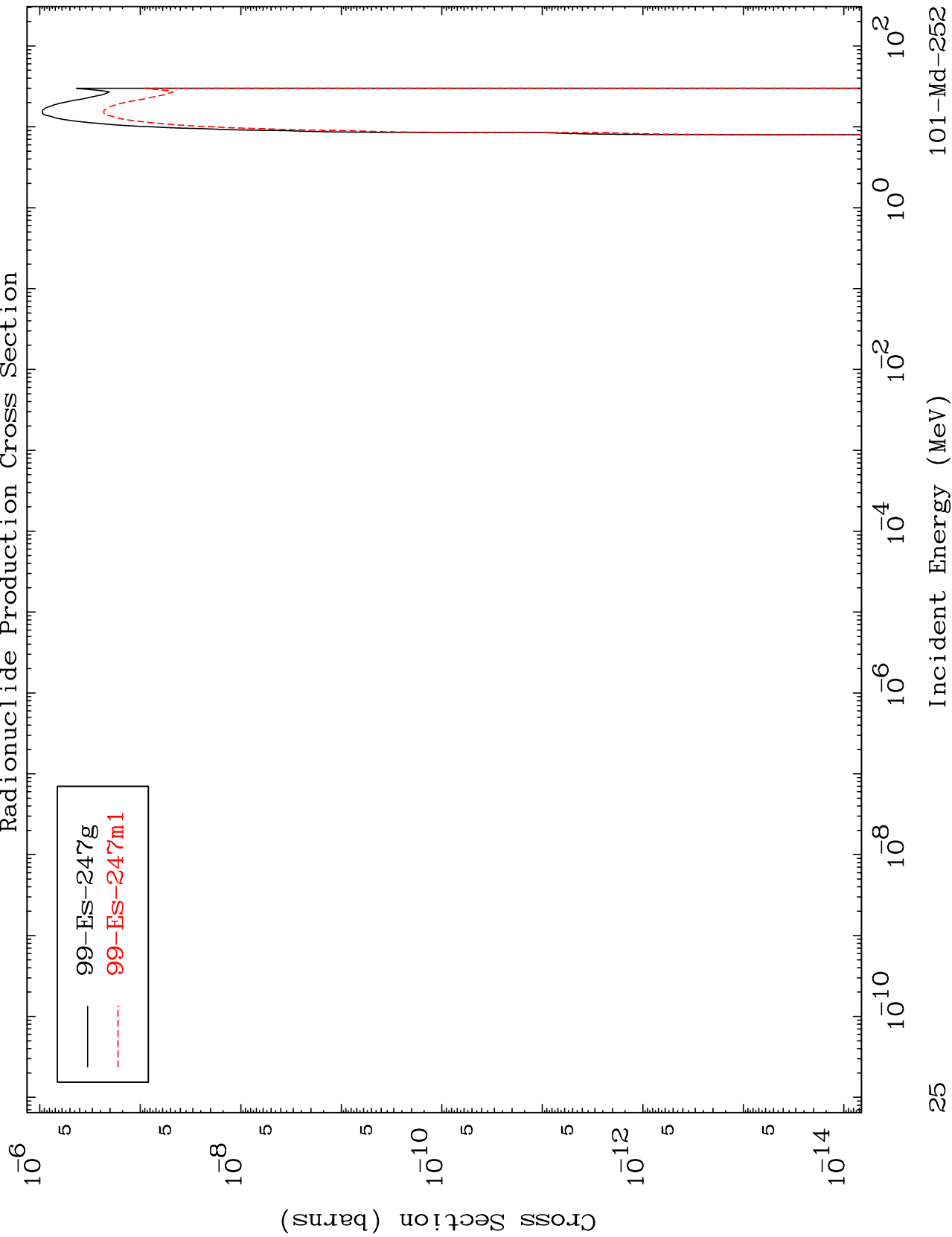


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(n,2n)  $\alpha$

101-Md-252

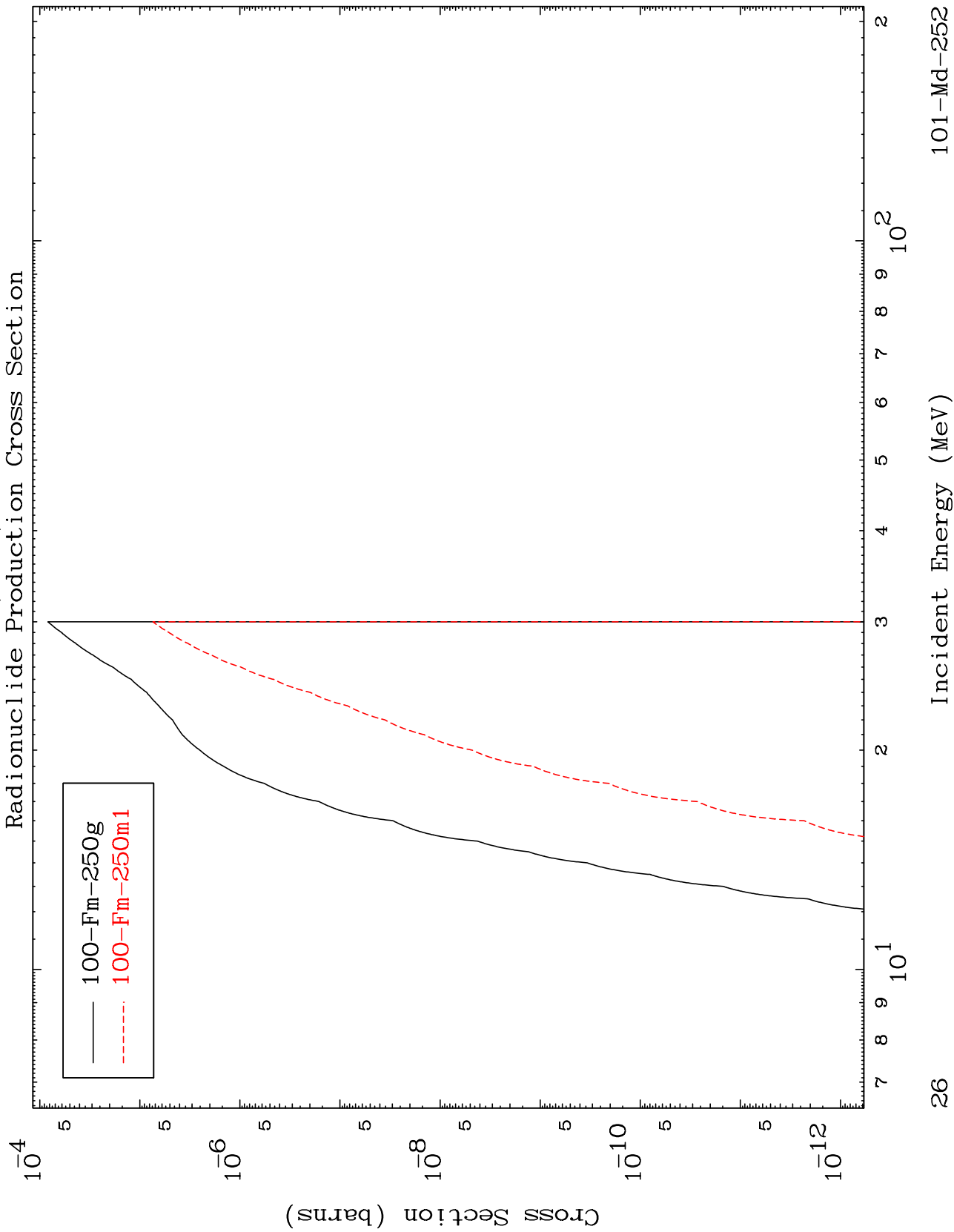
Radionuclide Production Cross Section



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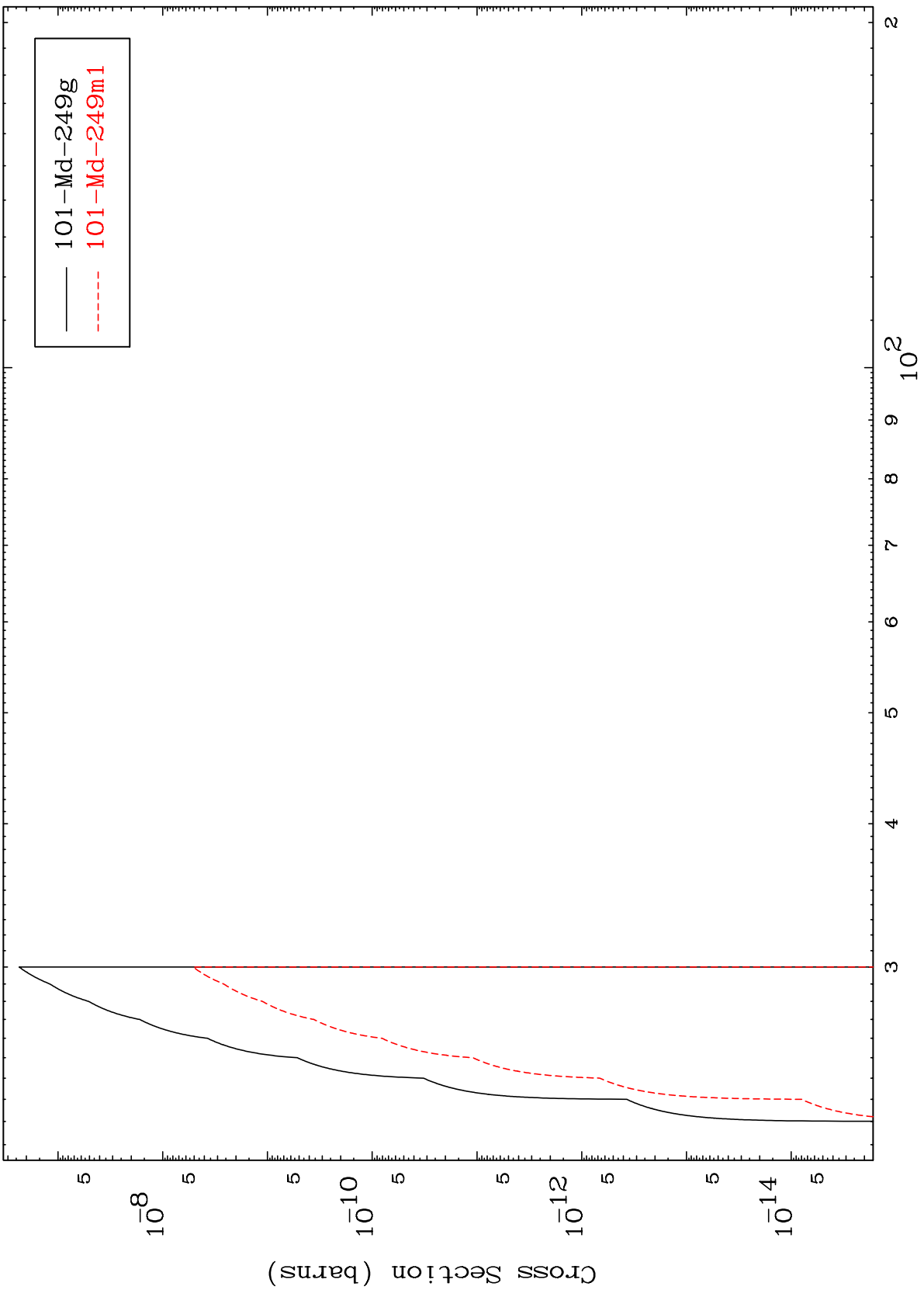
(n,n') d

101-Md-252



26

(n,4n)  
Radionuclide Production Cross Section

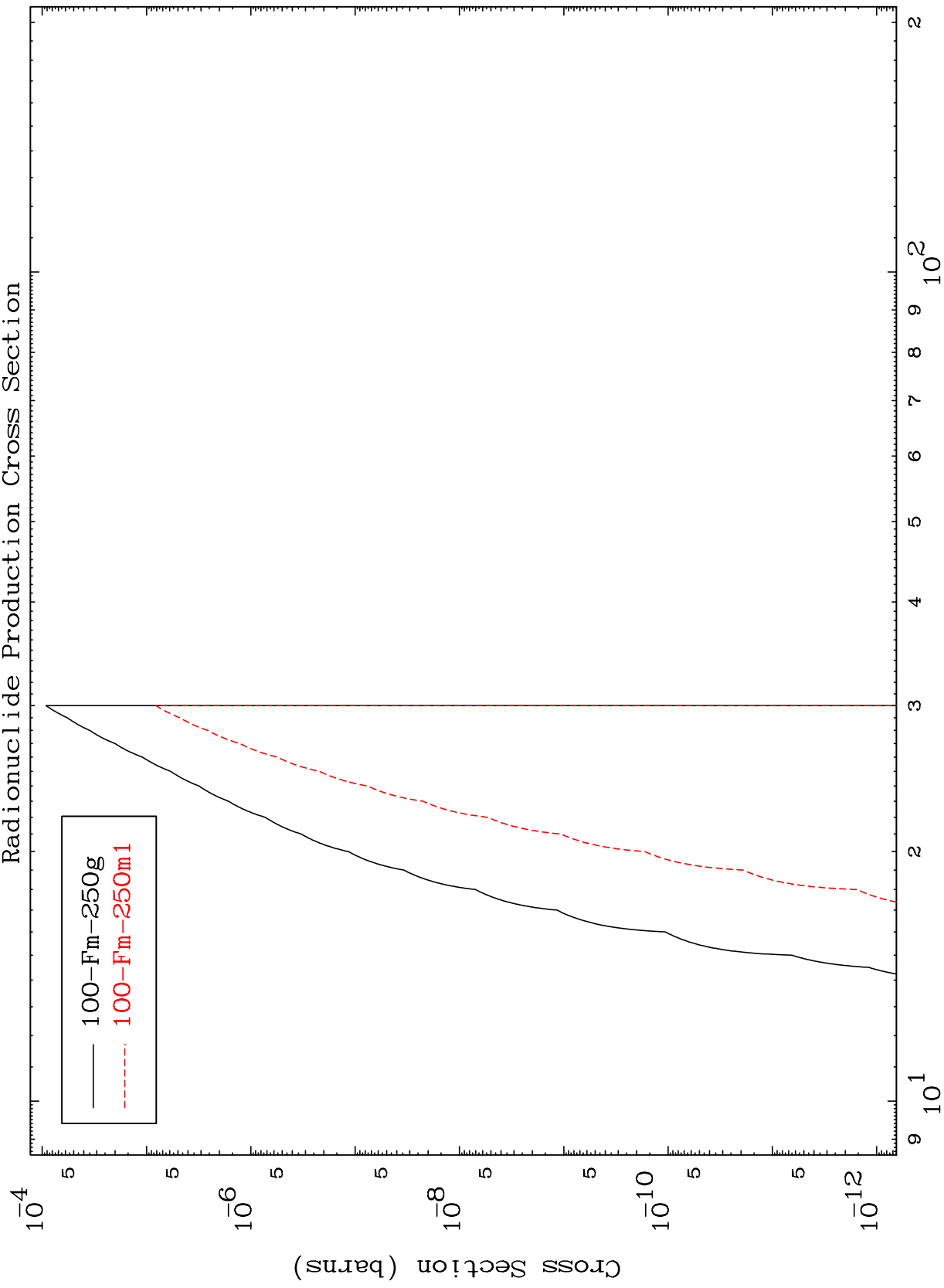


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(n,2n) p

101-Md-252

Radionuclide Production Cross Section



Incident Energy (MeV)

101-Md-252

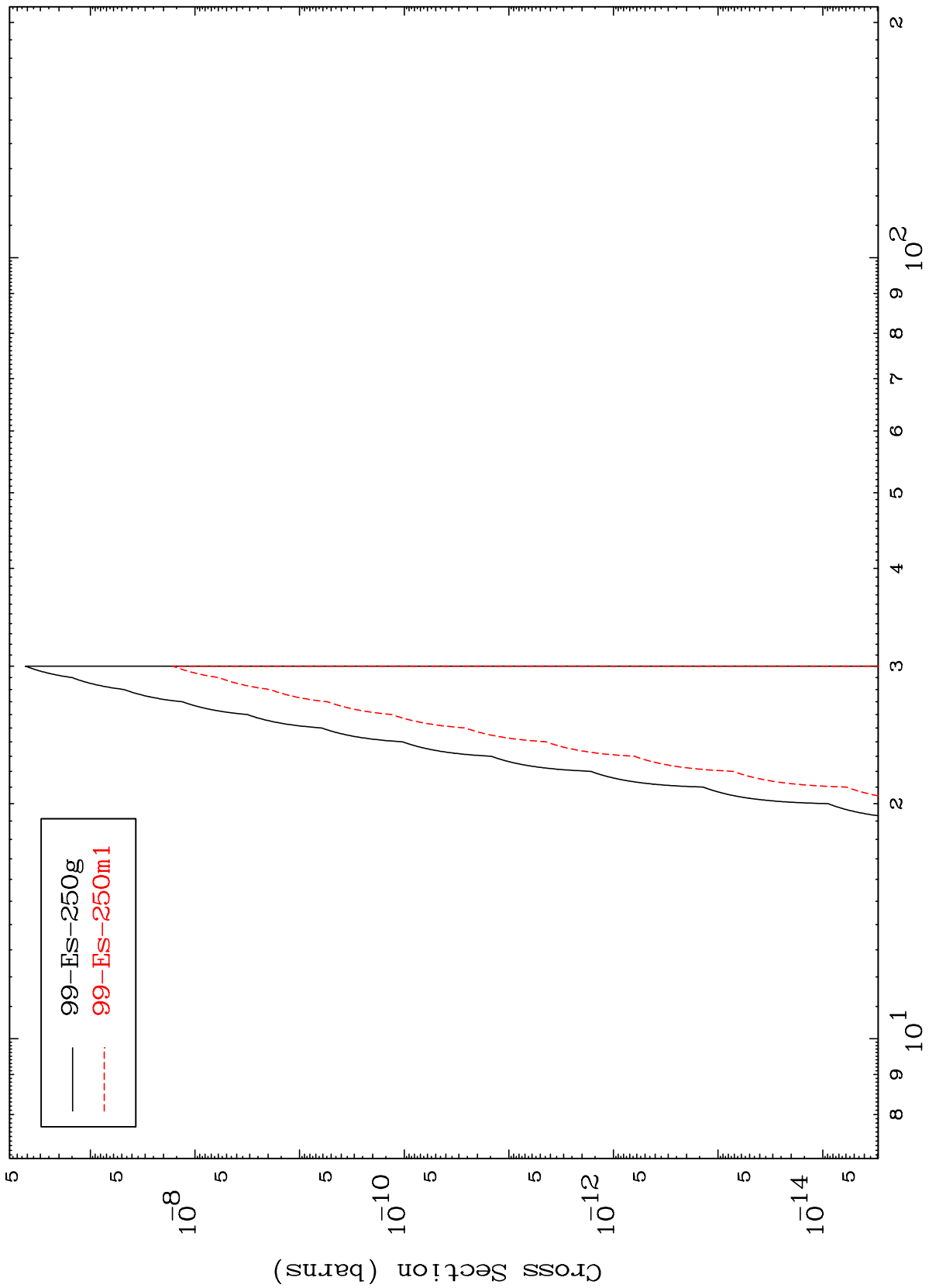
28

MAT 9953

(n,2n) p

101-Md-252

Radionuclide Production Cross Section



99-Es-250g  
99-Es-250m1

29

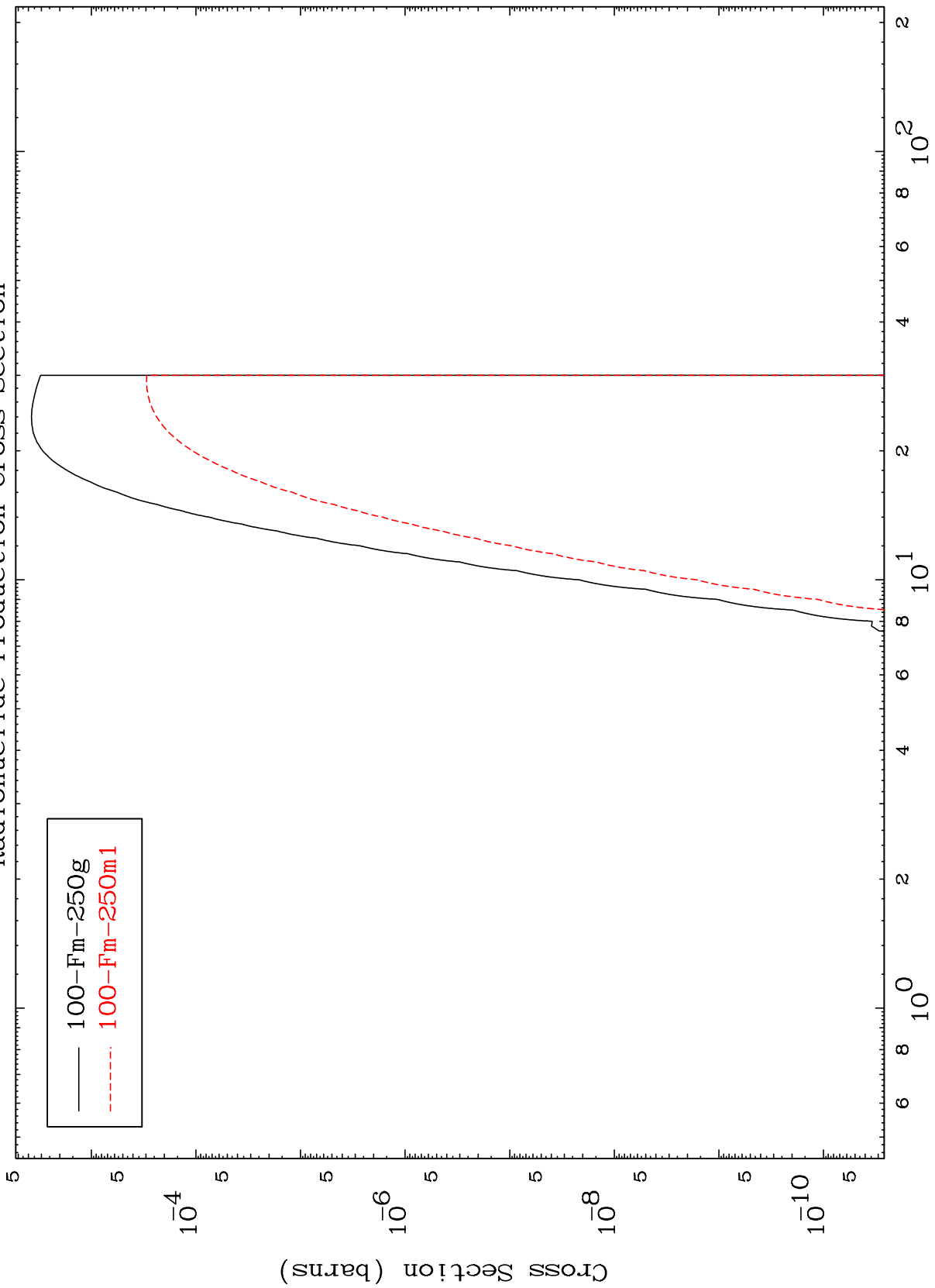
Incident Energy (MeV)

101-Md-252

MAT 9953

101-Md-252

(n, t)  
Radionuclide Production Cross Section



— 100-Fm-250g  
- - - 100-Fm-250m1

101-Md-252

Incident Energy (MeV)

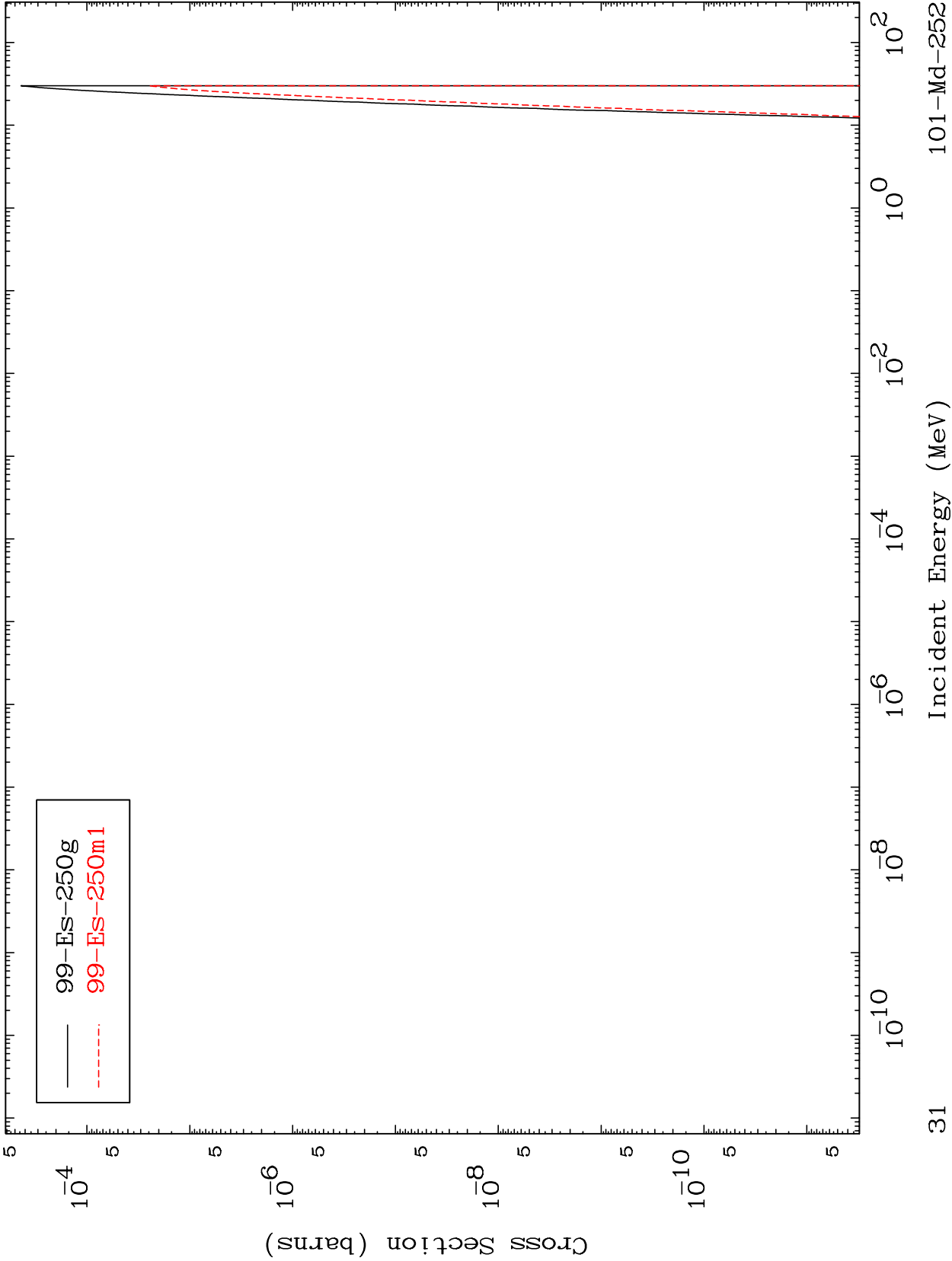
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(n,He-3)

101-Md-252

Radionuclide Production Cross Section





MAT 9953

(n,p) d

101-Md-252

