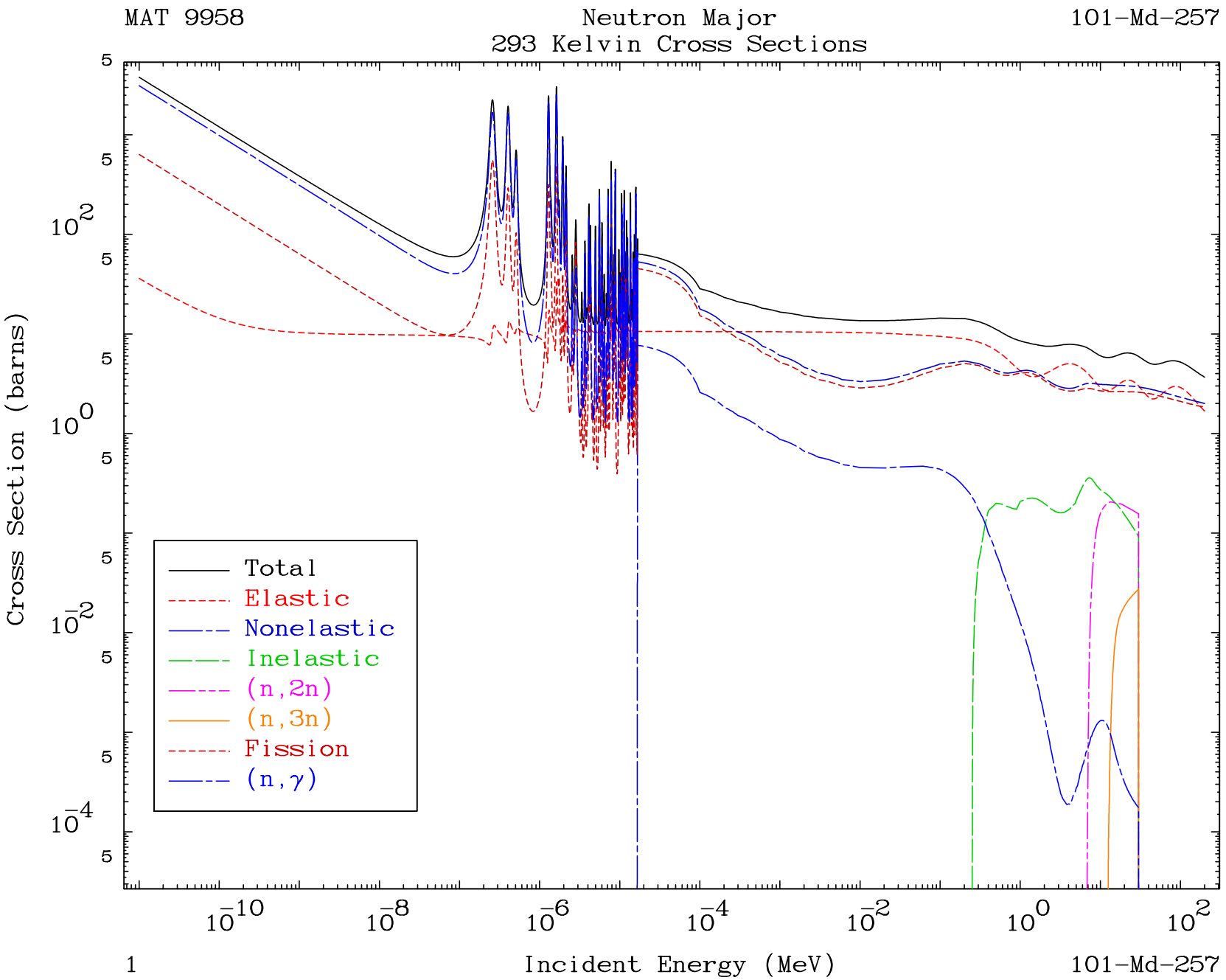
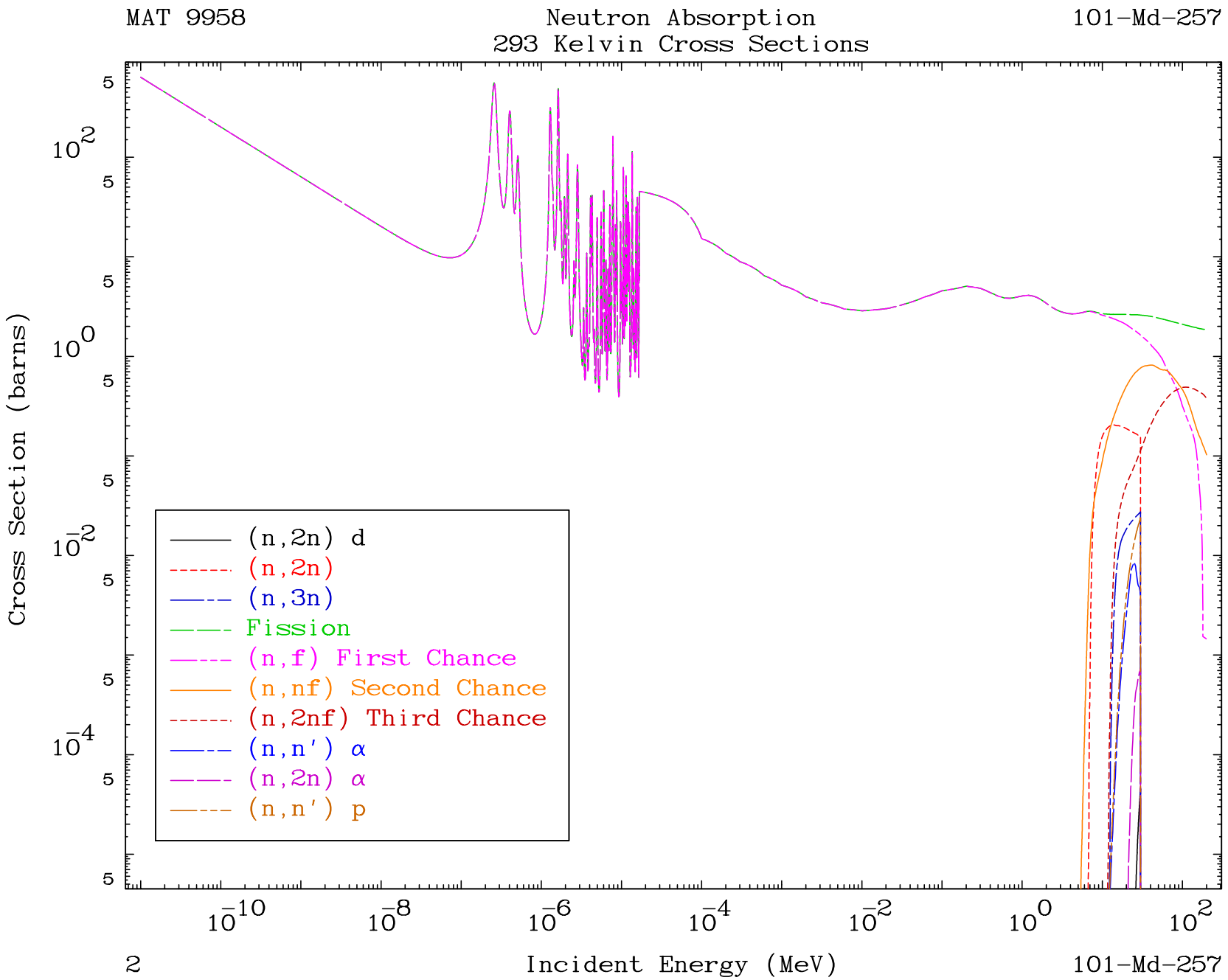
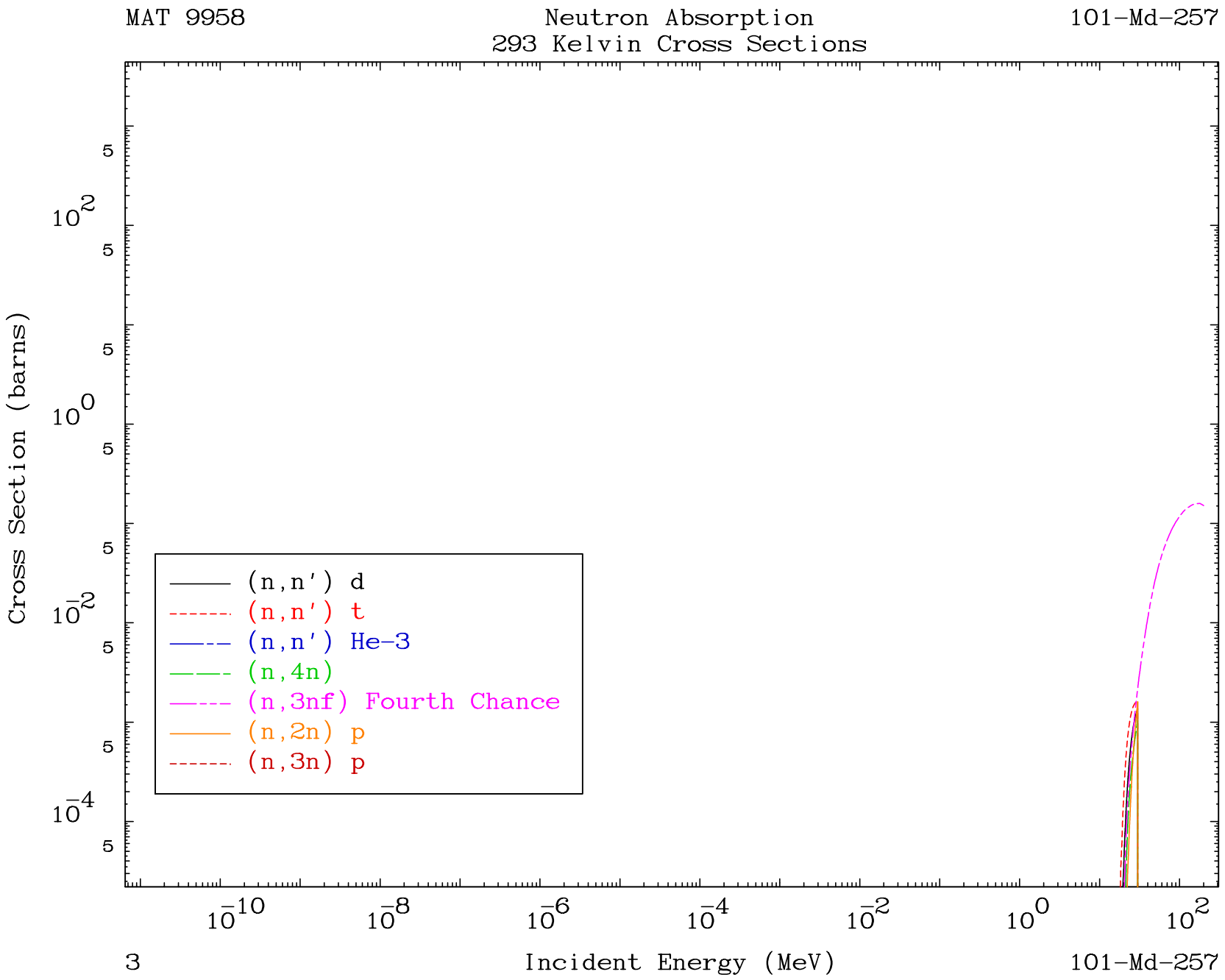
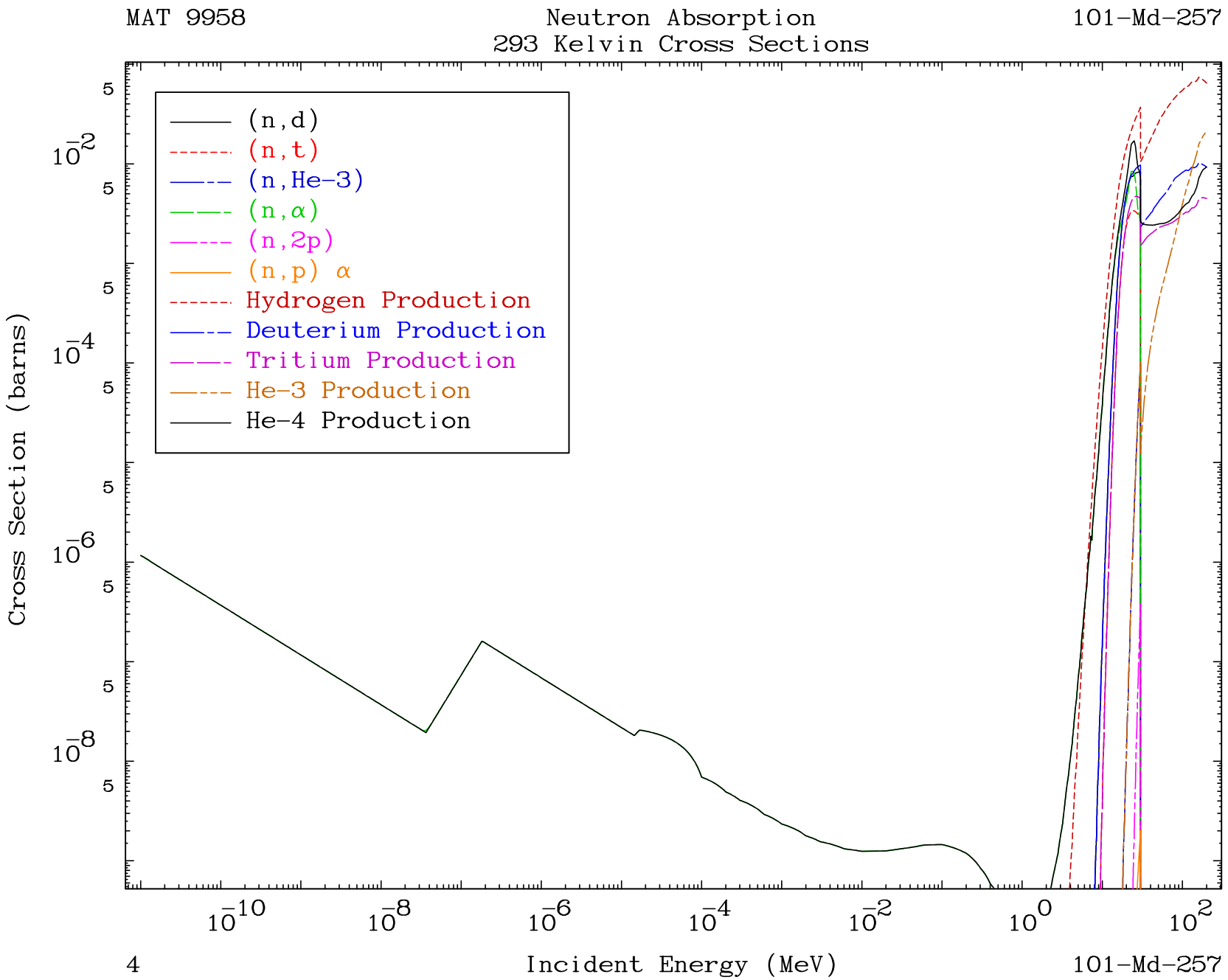
Program EVALPLOT
(Version 2021-1)
by
Dermott E. Cullen
(Present Contact Information)
Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.
Tele: 925-443-1911
E.Mail:redcullen1@comcast.net
Web:redcullen1.net/HOMEPAGE.NEW

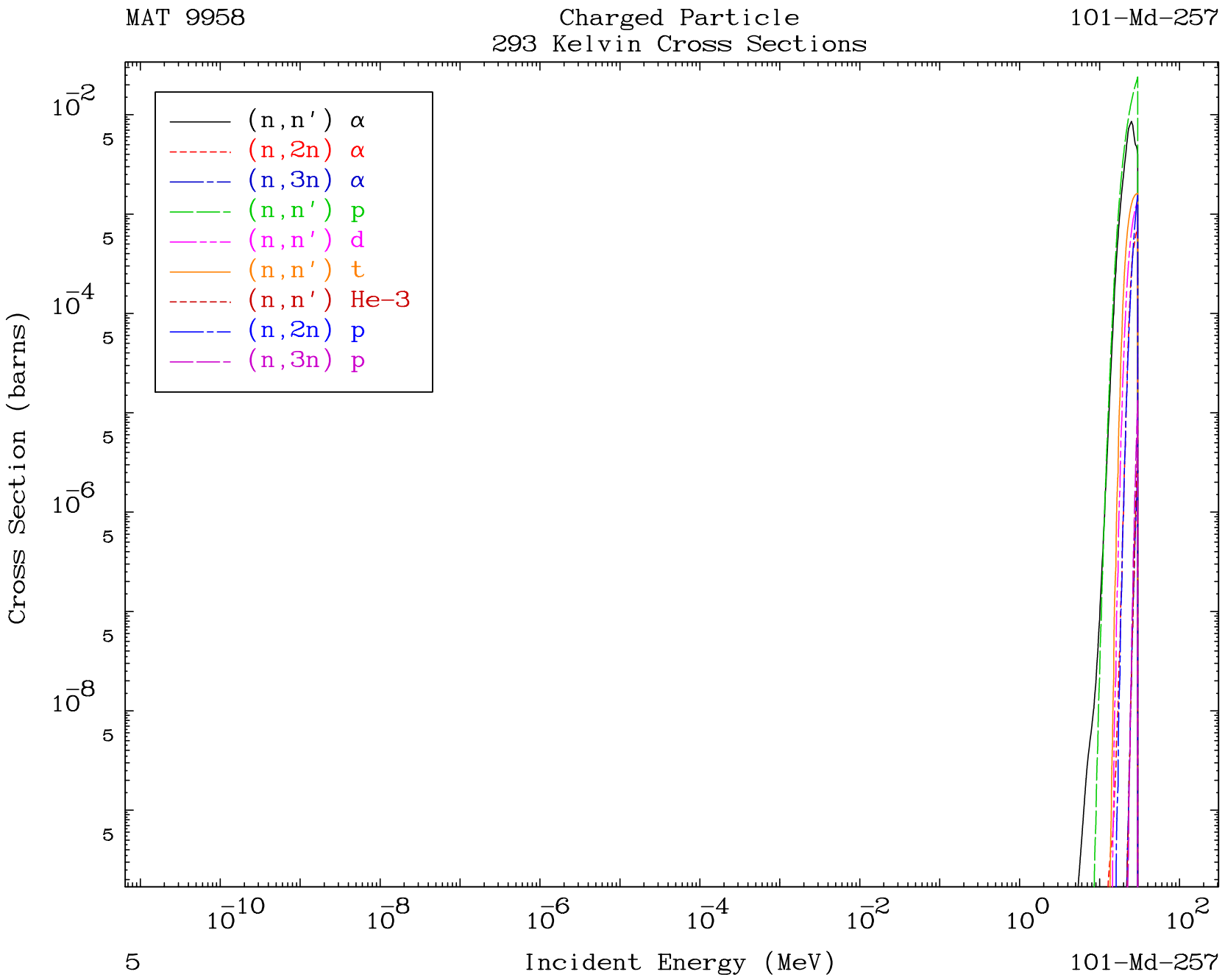
Press Mouse Button to Start

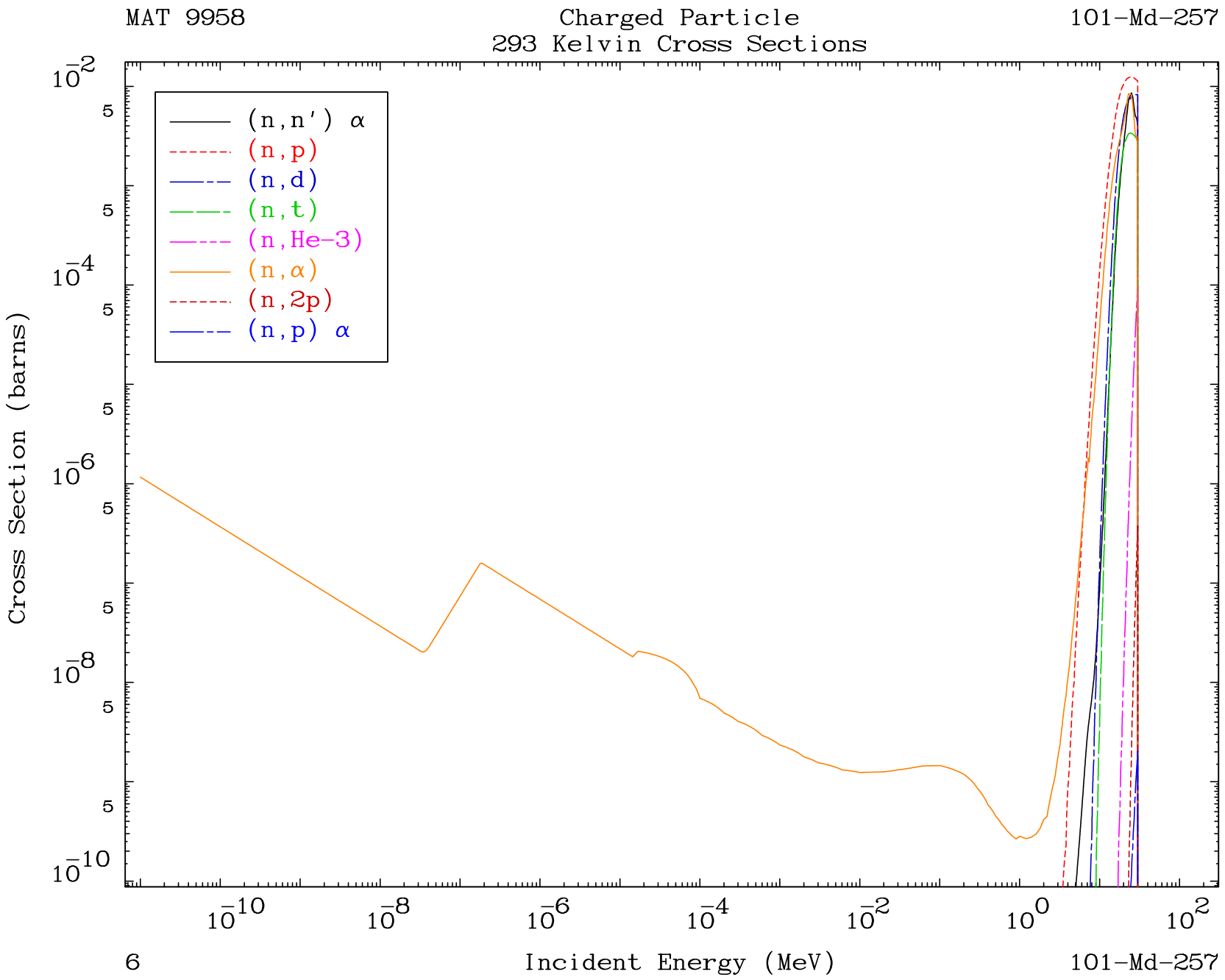

MAT 9958
Neutron Major
293 Kelvin Cross Sections
101-Md-257
Cross Section (barns)
Incident Energy (MeV)
Total
Elastic
Nonelastic
Inelastic
(n,2n)
(n,3n)
Fission
(n,γ)
1
101-Md-257

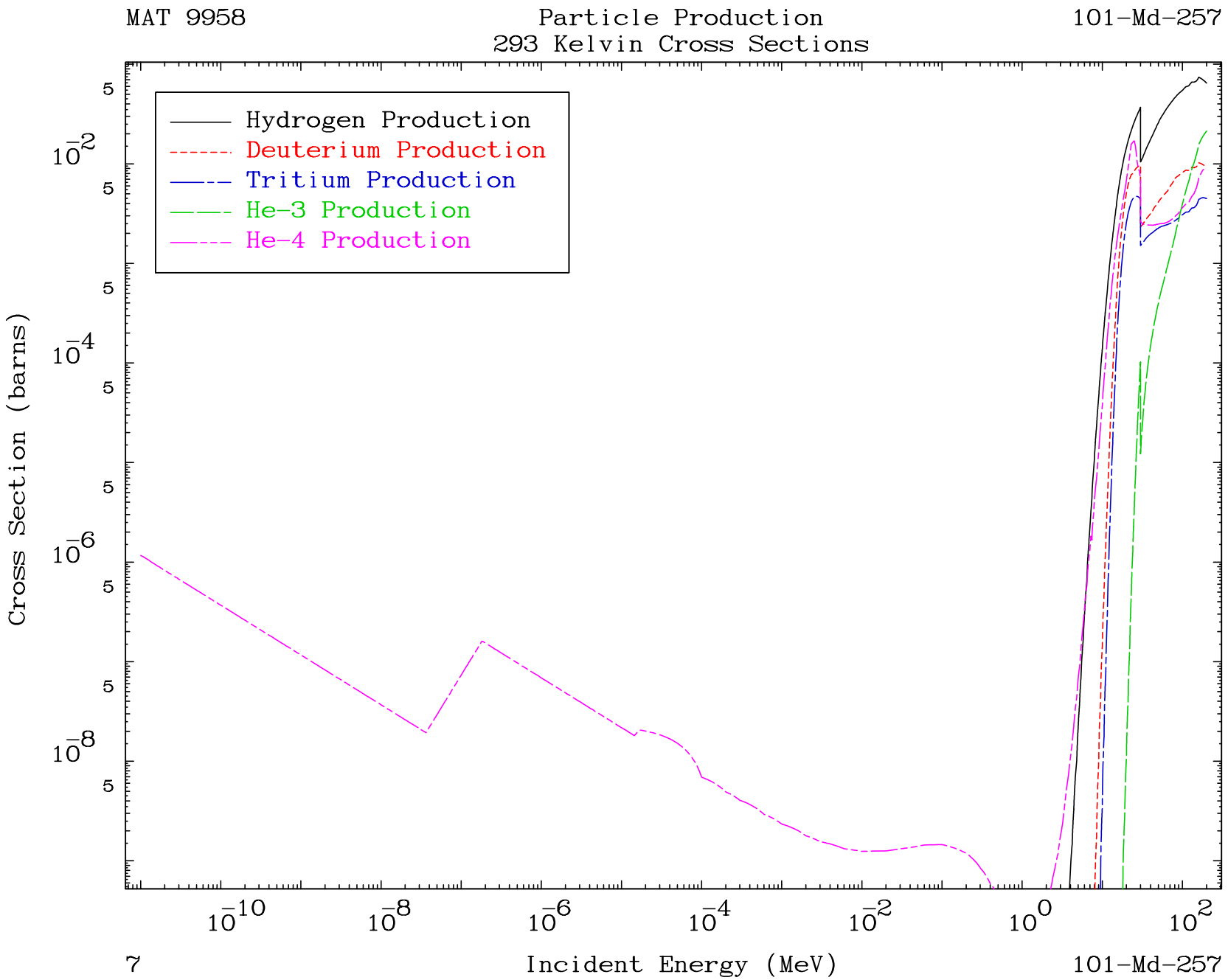

MAT 9958
Neutron Absorption
293 Kelvin Cross Sections
101-Md-257
Cross Section (barns)
Incident Energy (MeV)
(n,2n) d
(n,2n)
(n,3n)
Fission
(n,f) First Chance
(n,nf) Second Chance
(n,2nf) Third Chance
(n,n') α
(n,2n) α
(n,n') p
2
101-Md-257

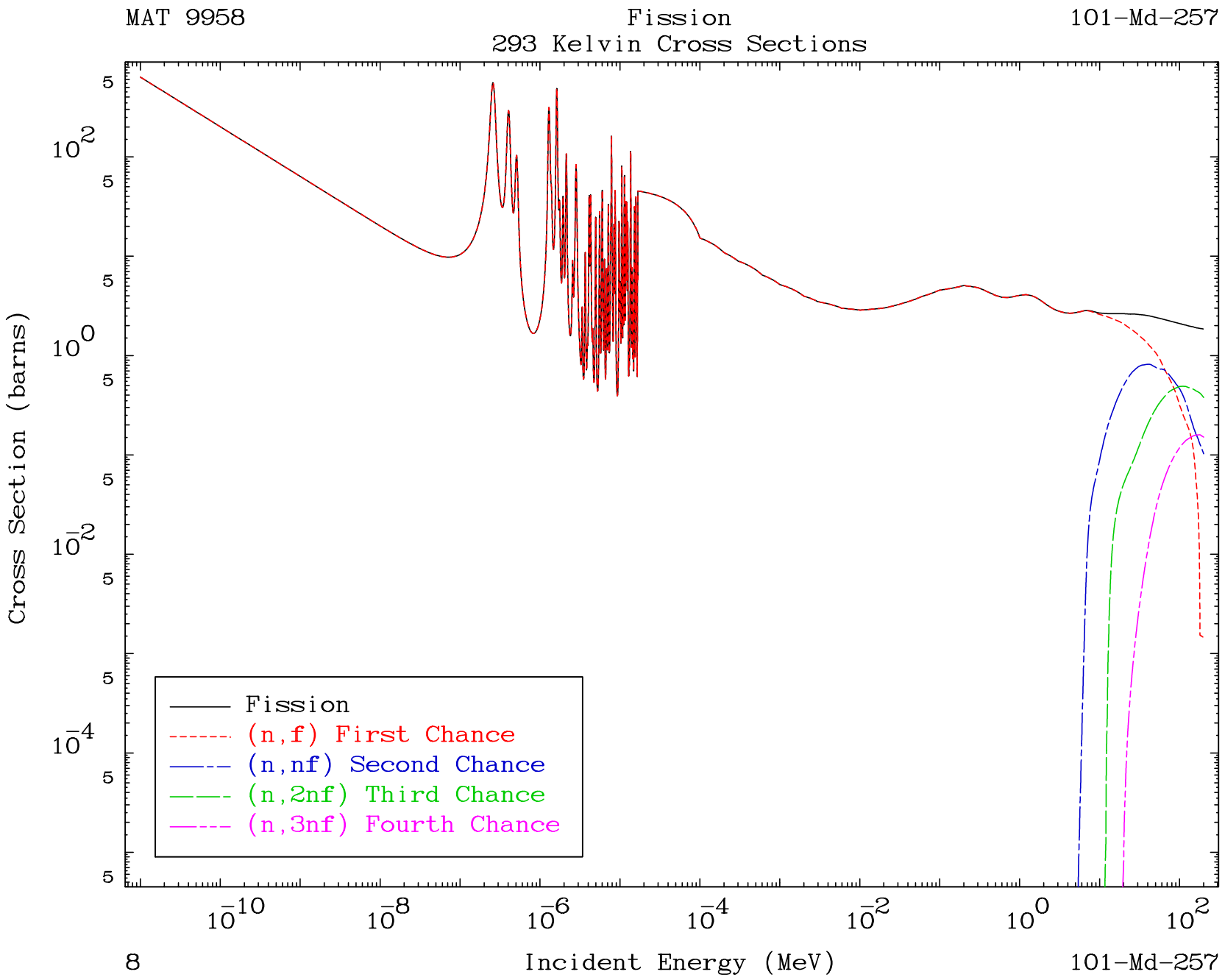

MAT 9958
Neutron Absorption
293 Kelvin Cross Sections
101-Md-257
Cross Section (barns)
10^2
10^0
10^-2
10^-4
5
(n,n') d
(n,n') t
(n,n') He-3
(n,4n)
(n,3nf) Fourth Chance
(n,2n) p
(n,3n) p
10^-10
10^-8
10^-6
10^-4
10^-2
10^0
10^2
Incident Energy (MeV)
3
101-Md-257

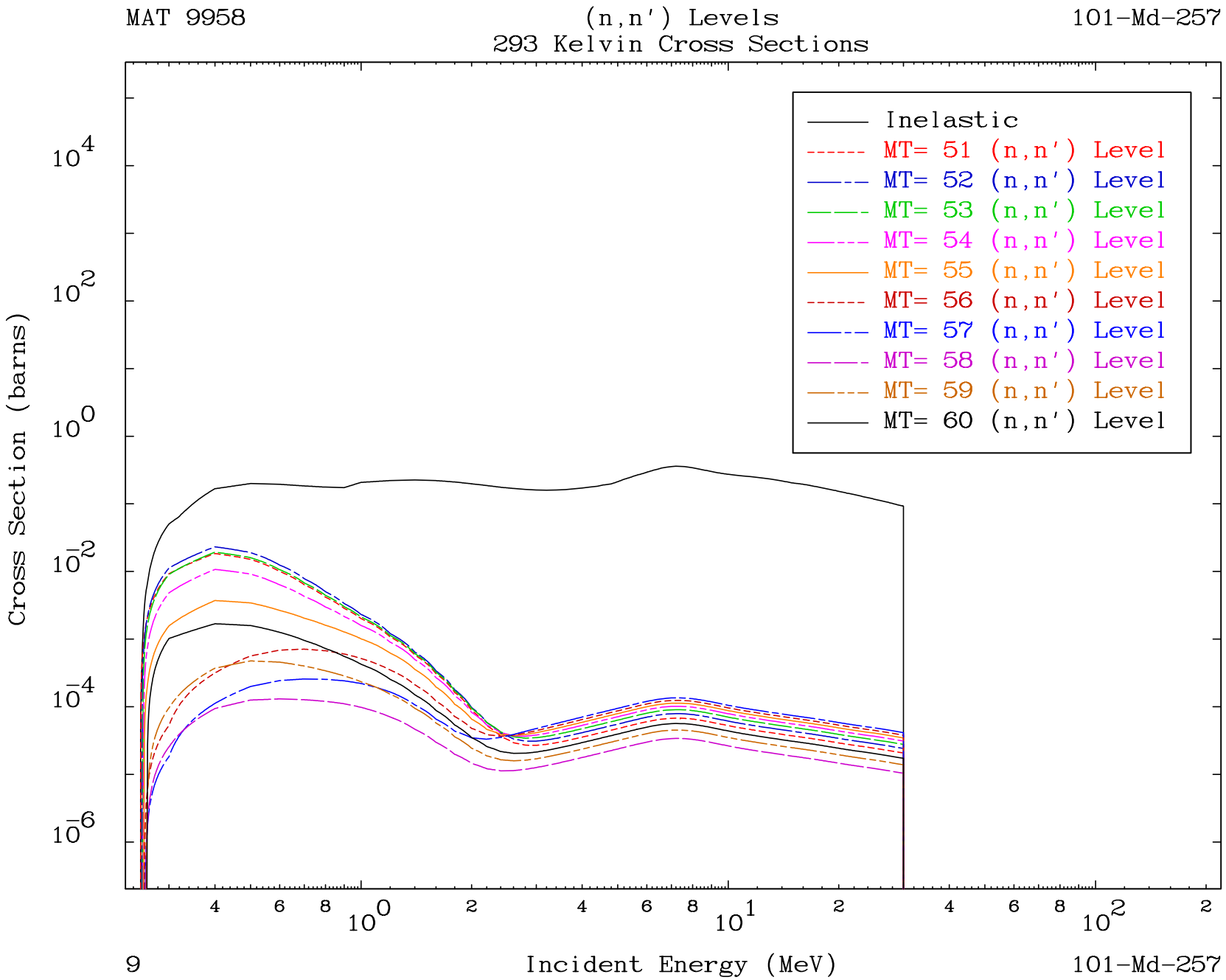

MAT 9958
Neutron Absorption
293 Kelvin Cross Sections
101-Md-257
(n,d)
(n,t)
(n,He-3)
(n,α)
(n,2p)
(n,p) α
Hydrogen Production
Deuterium Production
Tritium Production
He-3 Production
He-4 Production
Cross Section (barns)
Incident Energy (MeV)
4
101-Md-257

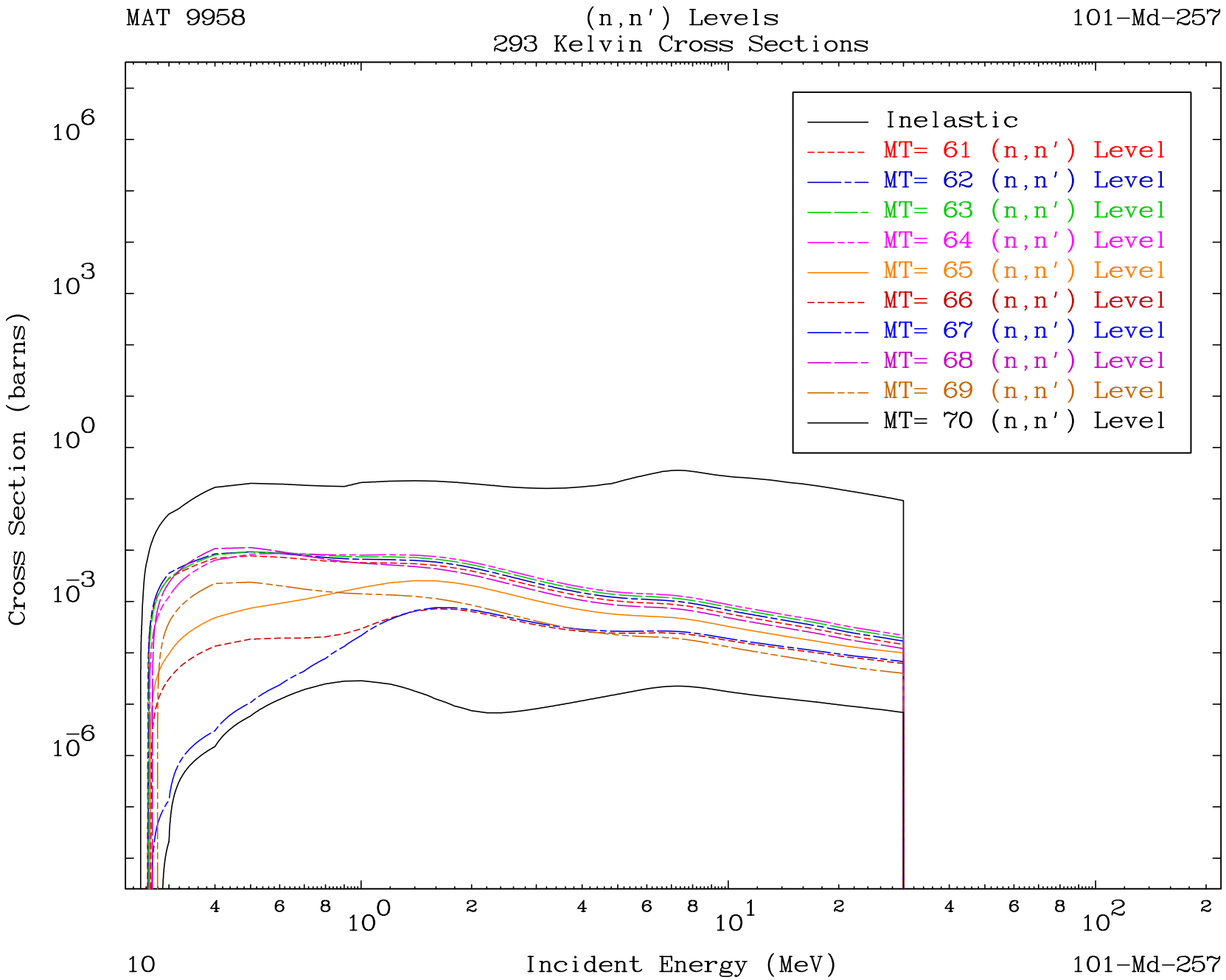

MAT 9958
Charged Particle
293 Kelvin Cross Sections
101-Md-257
(n,n') α
(n,2n) α
(n,3n) α
(n,n') p
(n,n') d
(n,n') t
(n,n') He-3
(n,2n) p
(n,3n) p
Cross Section (barns)
Incident Energy (MeV)
101-Md-257
5

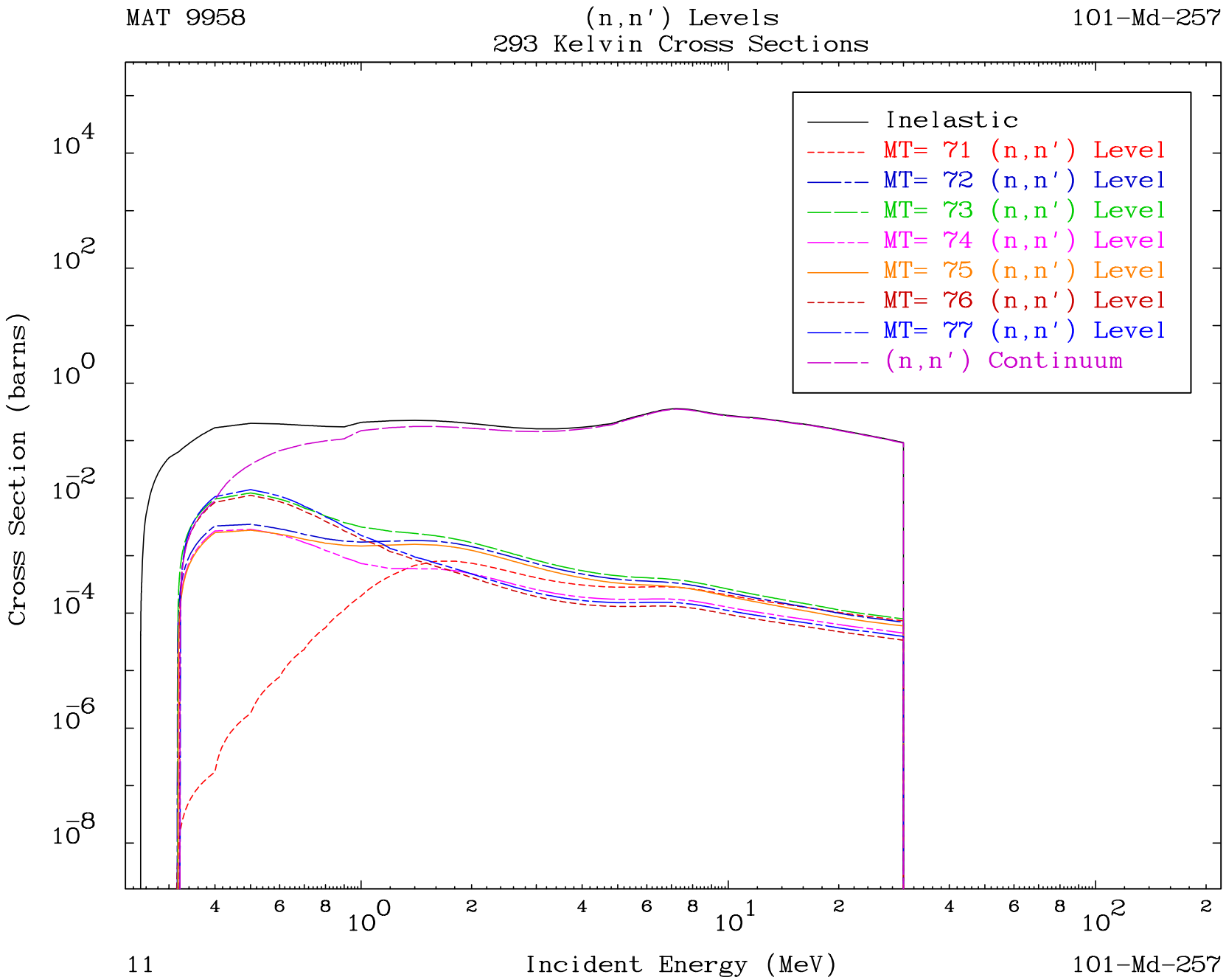

MAT 9958
Charged Particle
293 Kelvin Cross Sections
101-Md-257
(n,n') α
(n,p)
(n,d)
(n,t)
(n,He-3)
(n,α)
(n,2p)
(n,p) α
Cross Section (barns)
Incident Energy (MeV)
6
101-Md-257

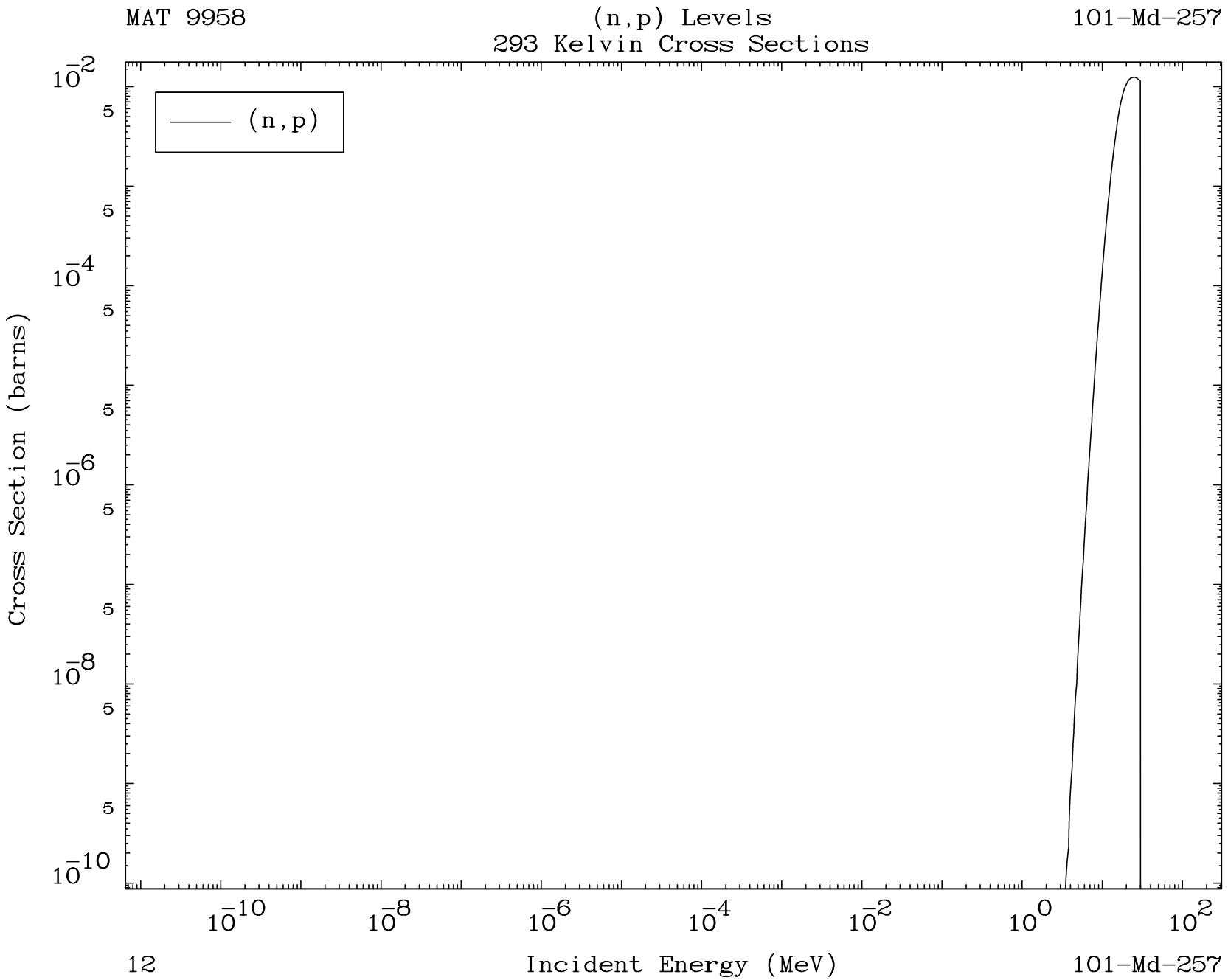

MAT 9958
Particle Production
293 Kelvin Cross Sections
101-Md-257
Hydrogen Production
Deuterium Production
Tritium Production
He-3 Production
He-4 Production
Cross Section (barns)
Incident Energy (MeV)
7
101-Md-257

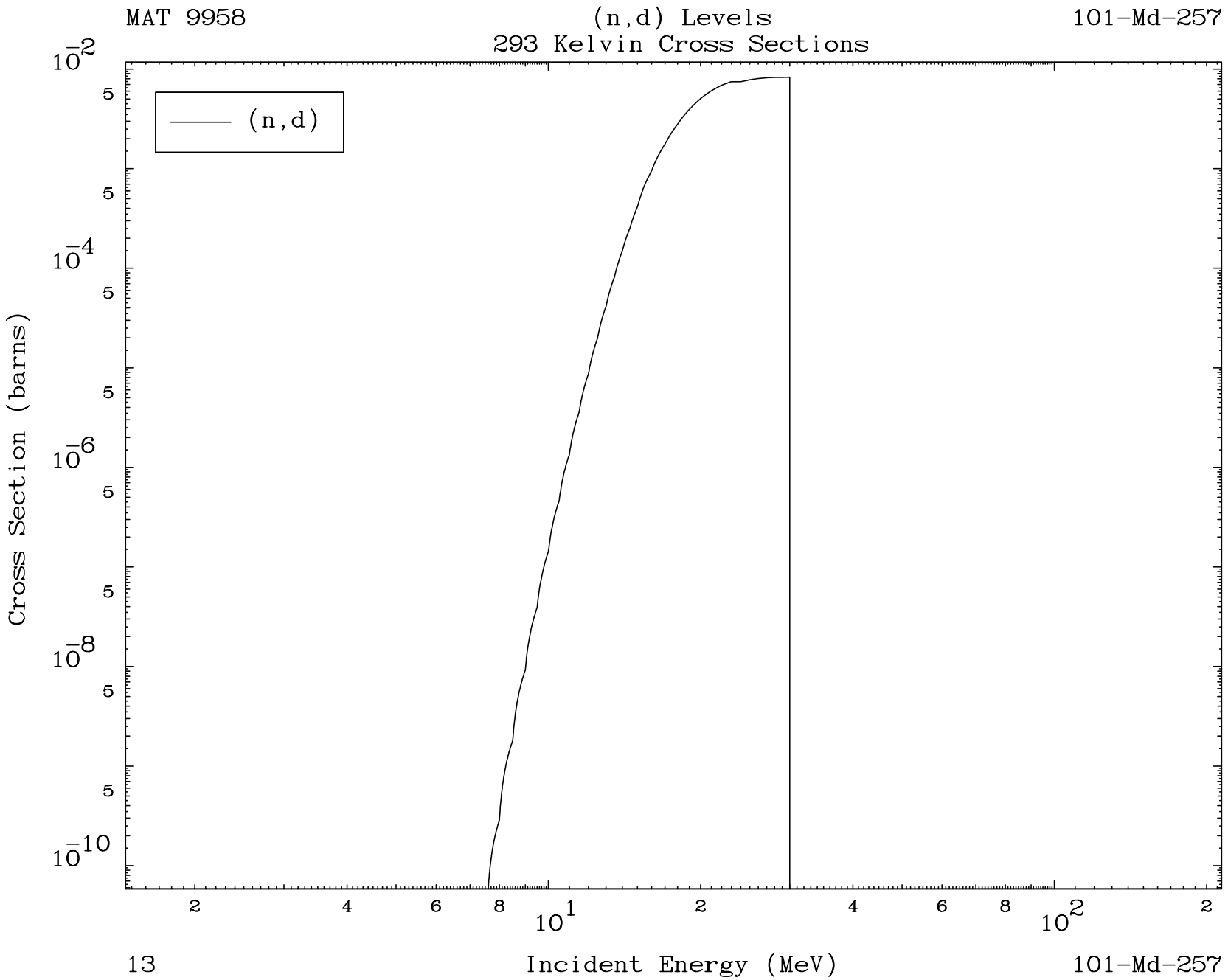

MAT 9958
Fission
101-Md-257
293 Kelvin Cross Sections
Cross Section (barns)
Incident Energy (MeV)
Fission
(n,f) First Chance
(n,nf) Second Chance
(n,2nf) Third Chance
(n,3nf) Fourth Chance
8
101-Md-257

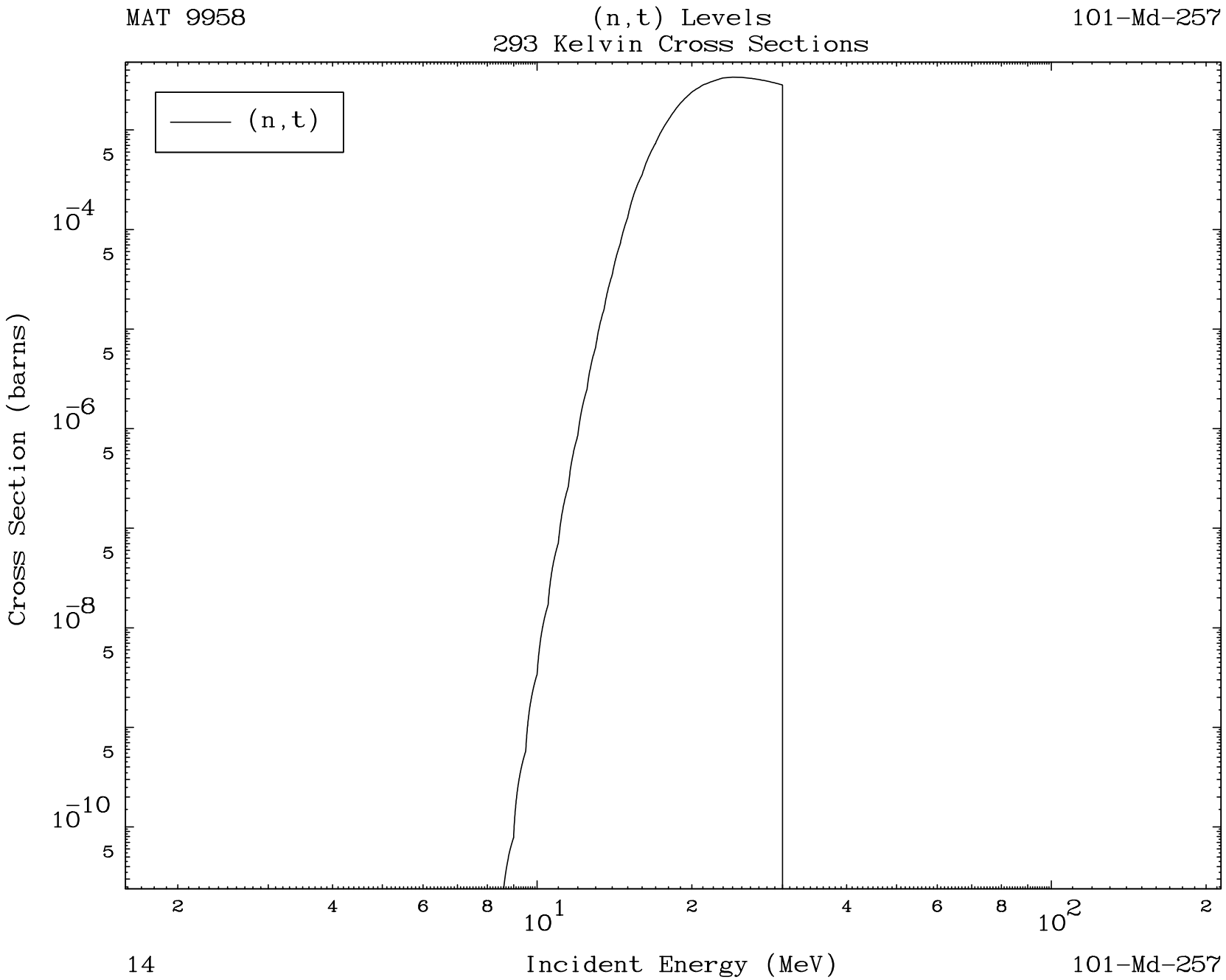

MAT 9958
(n,n') Levels
293 Kelvin Cross Sections
101-Md-257
Inelastic
MT= 51 (n,n') Level
MT= 52 (n,n') Level
MT= 53 (n,n') Level
MT= 54 (n,n') Level
MT= 55 (n,n') Level
MT= 56 (n,n') Level
MT= 57 (n,n') Level
MT= 58 (n,n') Level
MT= 59 (n,n') Level
MT= 60 (n,n') Level
Cross Section (barns)
Incident Energy (MeV)
9
101-Md-257

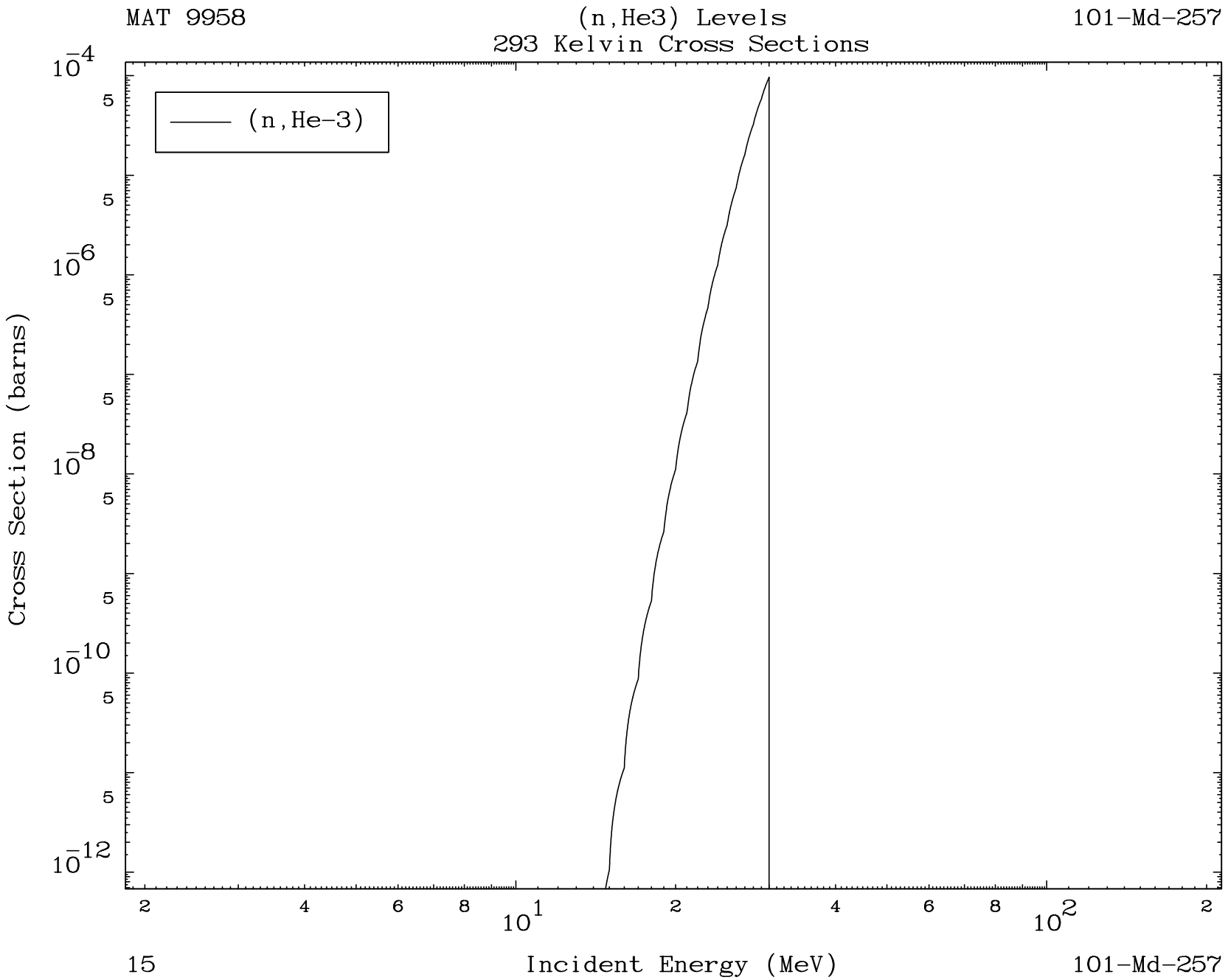

MAT 9958
(n,n') Levels
293 Kelvin Cross Sections
101-Md-257
Inelastic
MT= 61 (n,n') Level
MT= 62 (n,n') Level
MT= 63 (n,n') Level
MT= 64 (n,n') Level
MT= 65 (n,n') Level
MT= 66 (n,n') Level
MT= 67 (n,n') Level
MT= 68 (n,n') Level
MT= 69 (n,n') Level
MT= 70 (n,n') Level
Cross Section (barns)
Incident Energy (MeV)
10
101-Md-257

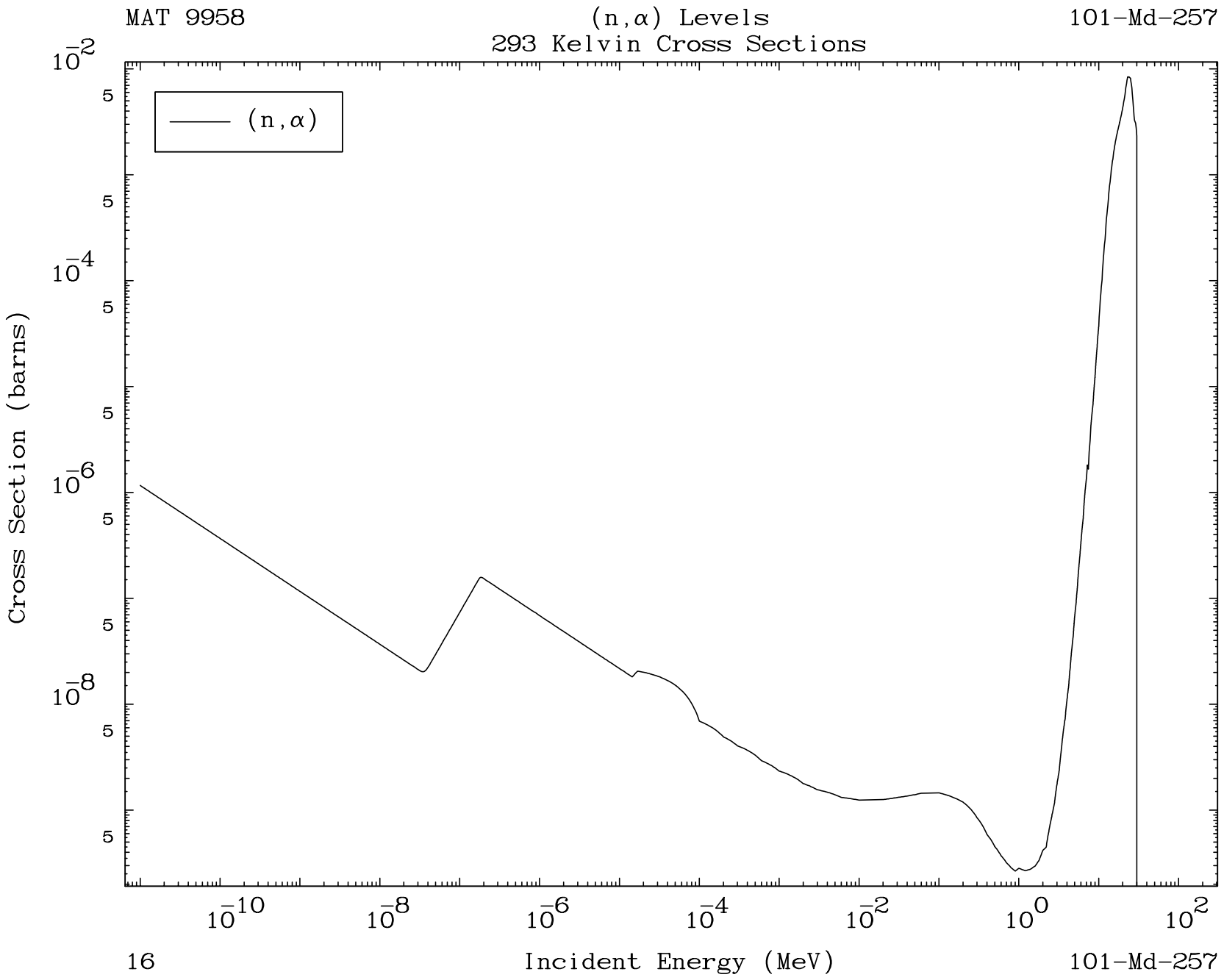

MAT 9958
(n,n') Levels
293 Kelvin Cross Sections
101-Md-257
Inelastic
MT= 71 (n,n') Level
MT= 72 (n,n') Level
MT= 73 (n,n') Level
MT= 74 (n,n') Level
MT= 75 (n,n') Level
MT= 76 (n,n') Level
MT= 77 (n,n') Level
(n,n') Continuum
Cross Section (barns)
Incident Energy (MeV)
11
101-Md-257

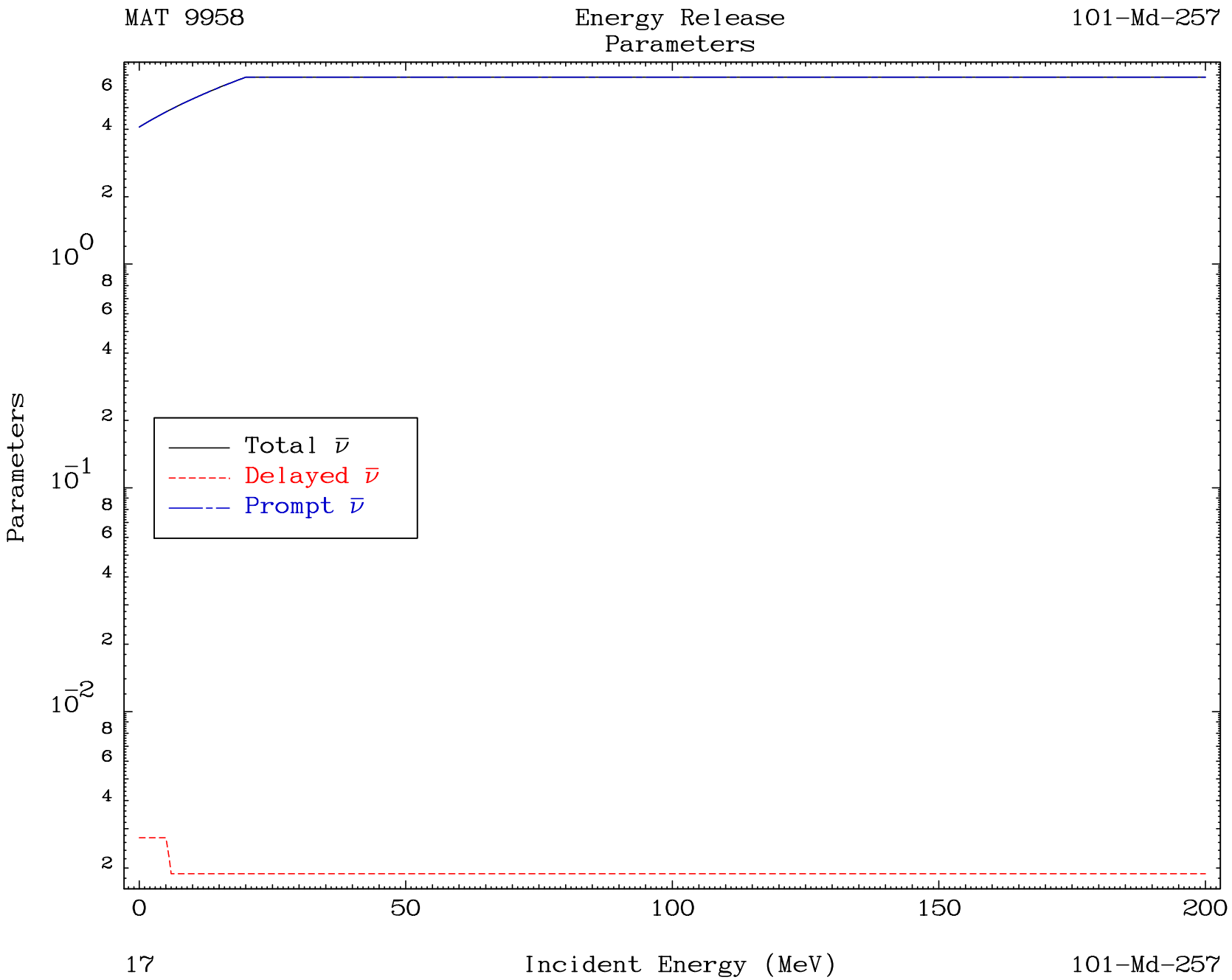

MAT 9958
(n,p) Levels
101-Md-257
293 Kelvin Cross Sections
(n,p)
Cross Section (barns)
Incident Energy (MeV)
12
101-Md-257

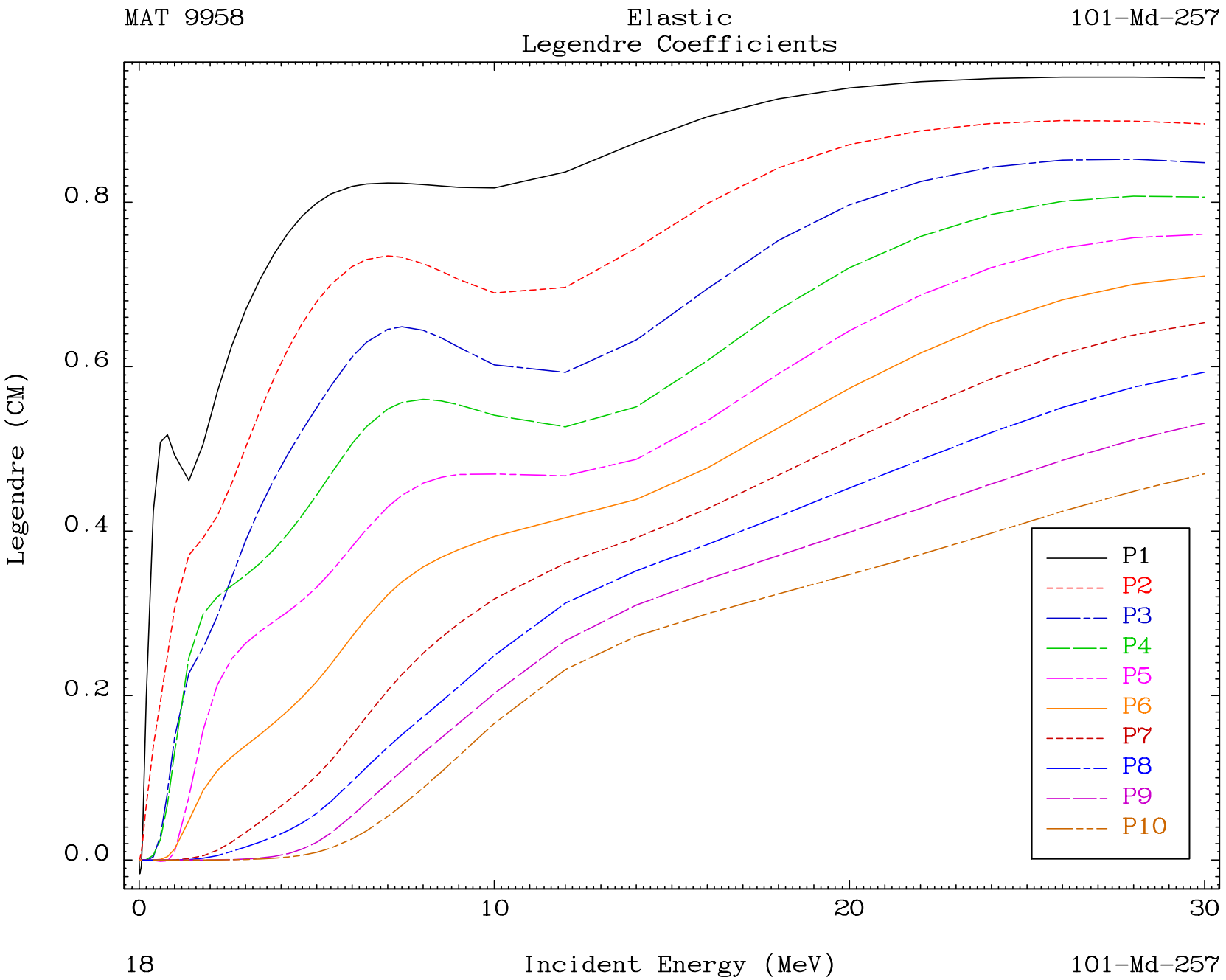

MAT 9958
(n,d) Levels
101-Md-257
293 Kelvin Cross Sections
(n,d)
Cross Section (barns)
Incident Energy (MeV)
13
101-Md-257

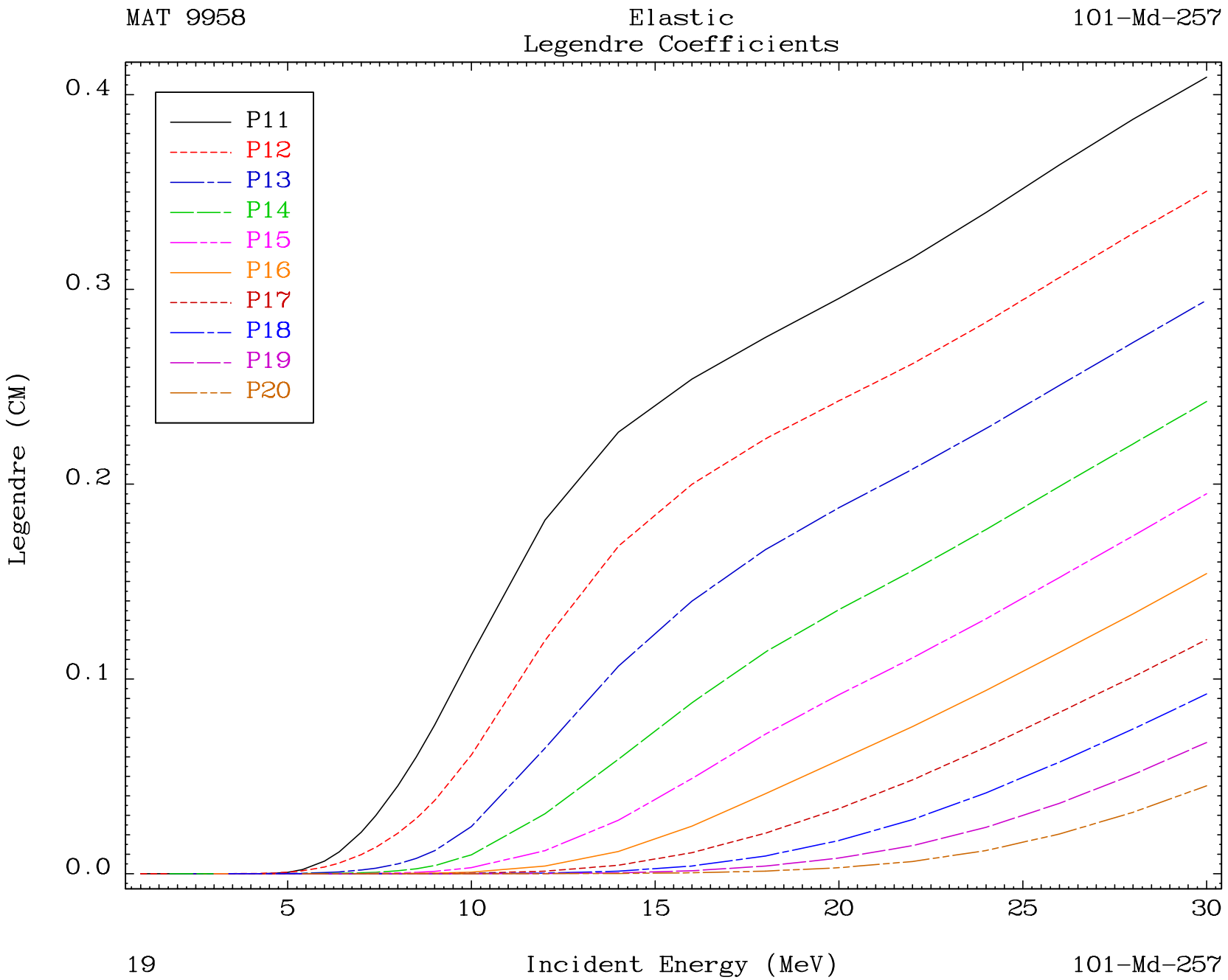

MAT 9958
(n,t) Levels
101-Md-257
293 Kelvin Cross Sections
(n,t)
Cross Section (barns)
10^-4
10^-6
10^-8
10^-10
10^1
10^2
Incident Energy (MeV)
14
101-Md-257

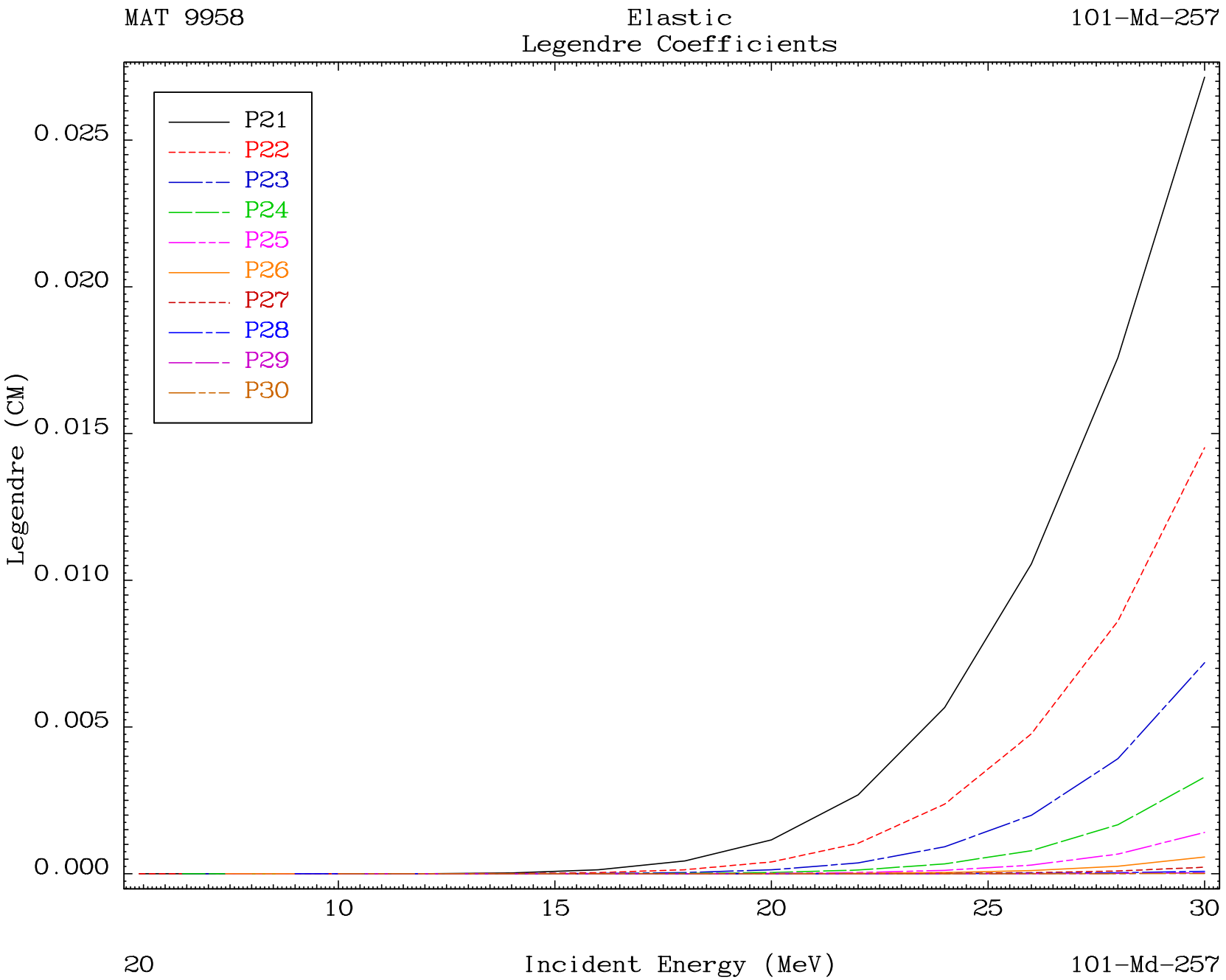

MAT 9958
(n,He3) Levels
101-Md-257
293 Kelvin Cross Sections
(n,He-3)
Cross Section (barns)
Incident Energy (MeV)
15
101-Md-257

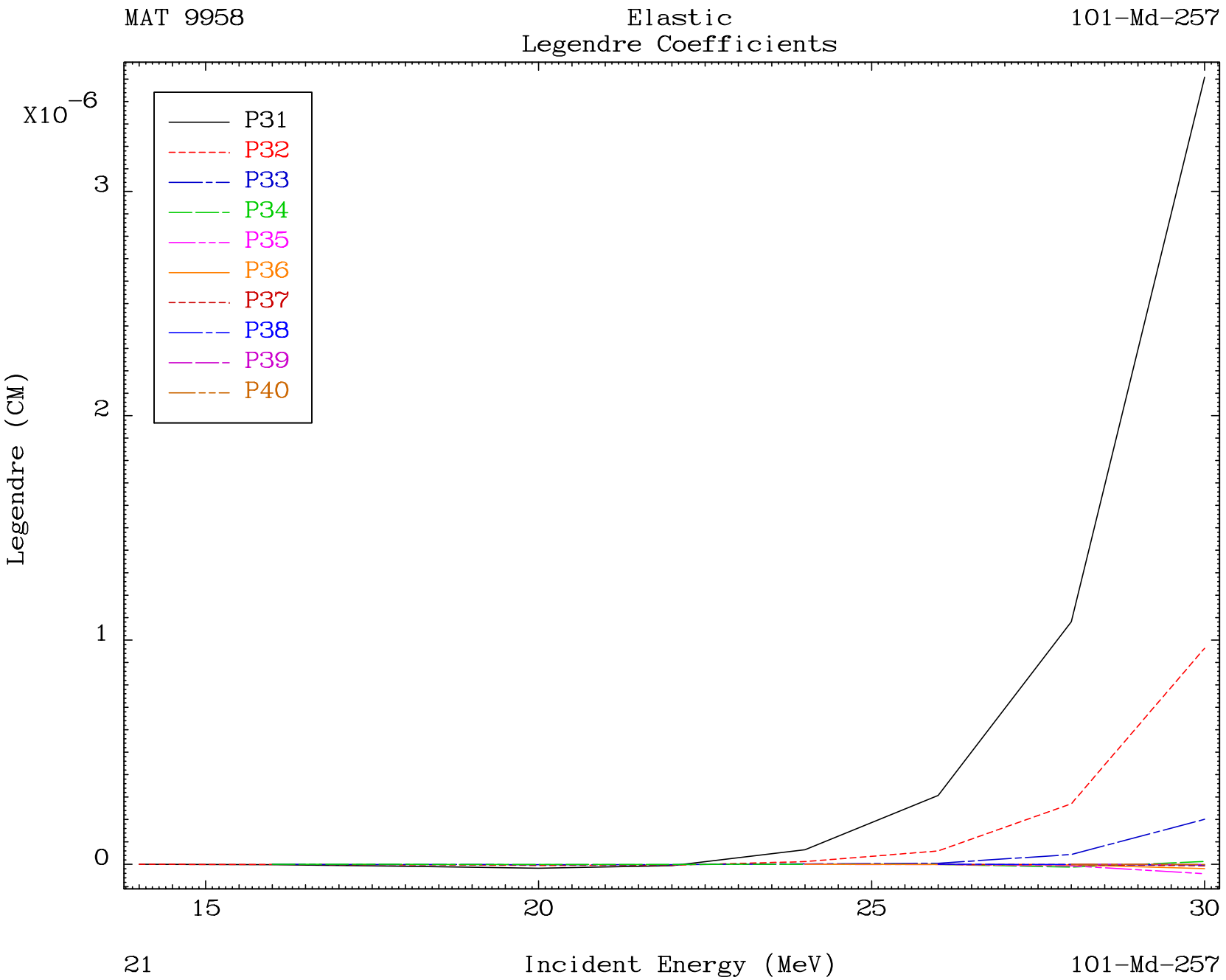

MAT 9958
(n,α) Levels
101-Md-257
293 Kelvin Cross Sections
(n,α)
Cross Section (barns)
Incident Energy (MeV)
16
101-Md-257

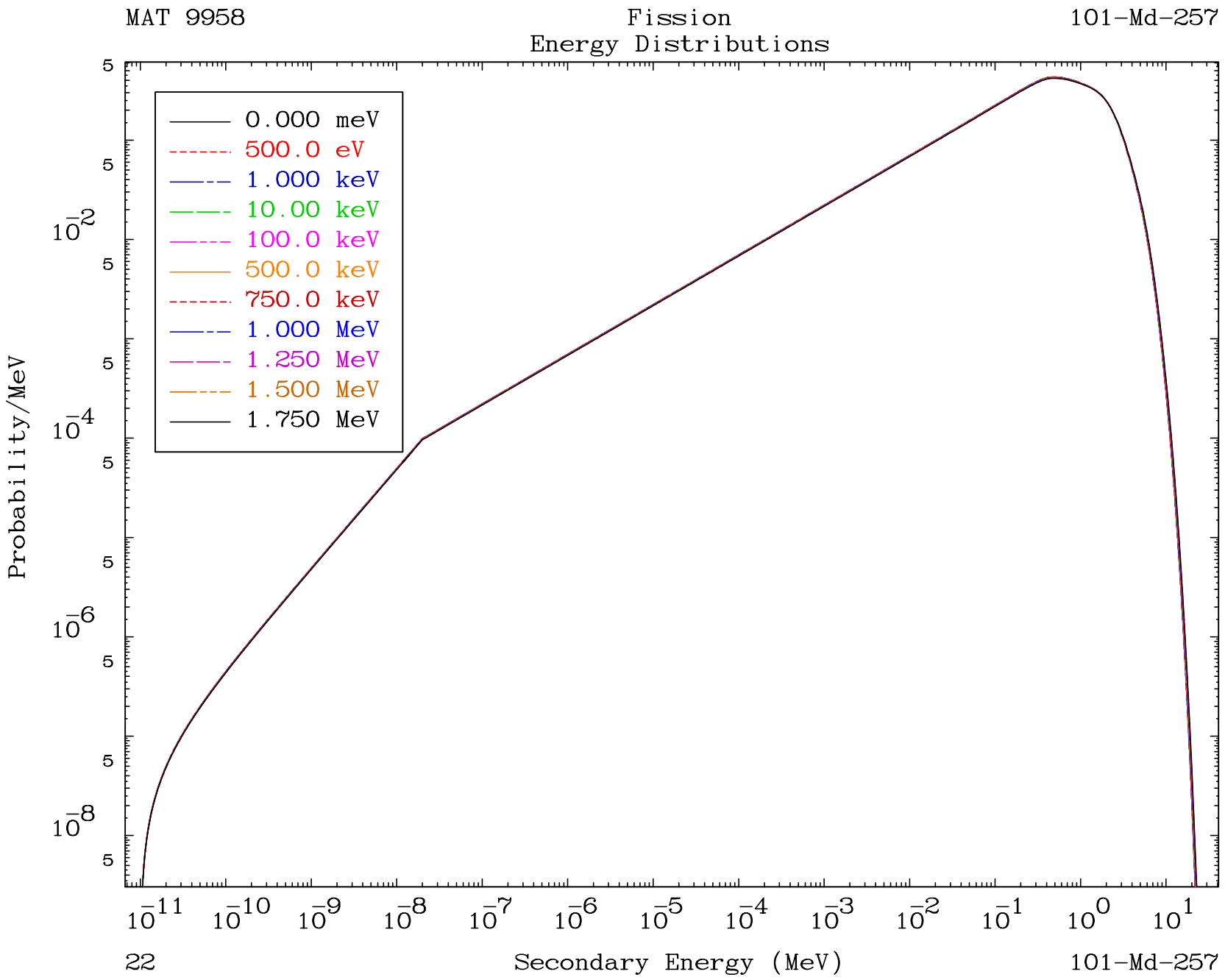

MAT 9958
Energy Release Parameters
101-Md-257
Parameters
Total $\bar{\nu}$
Delayed $\bar{\nu}$
Prompt $\bar{\nu}$
0
50
100
150
200
Incident Energy (MeV)
17
101-Md-257

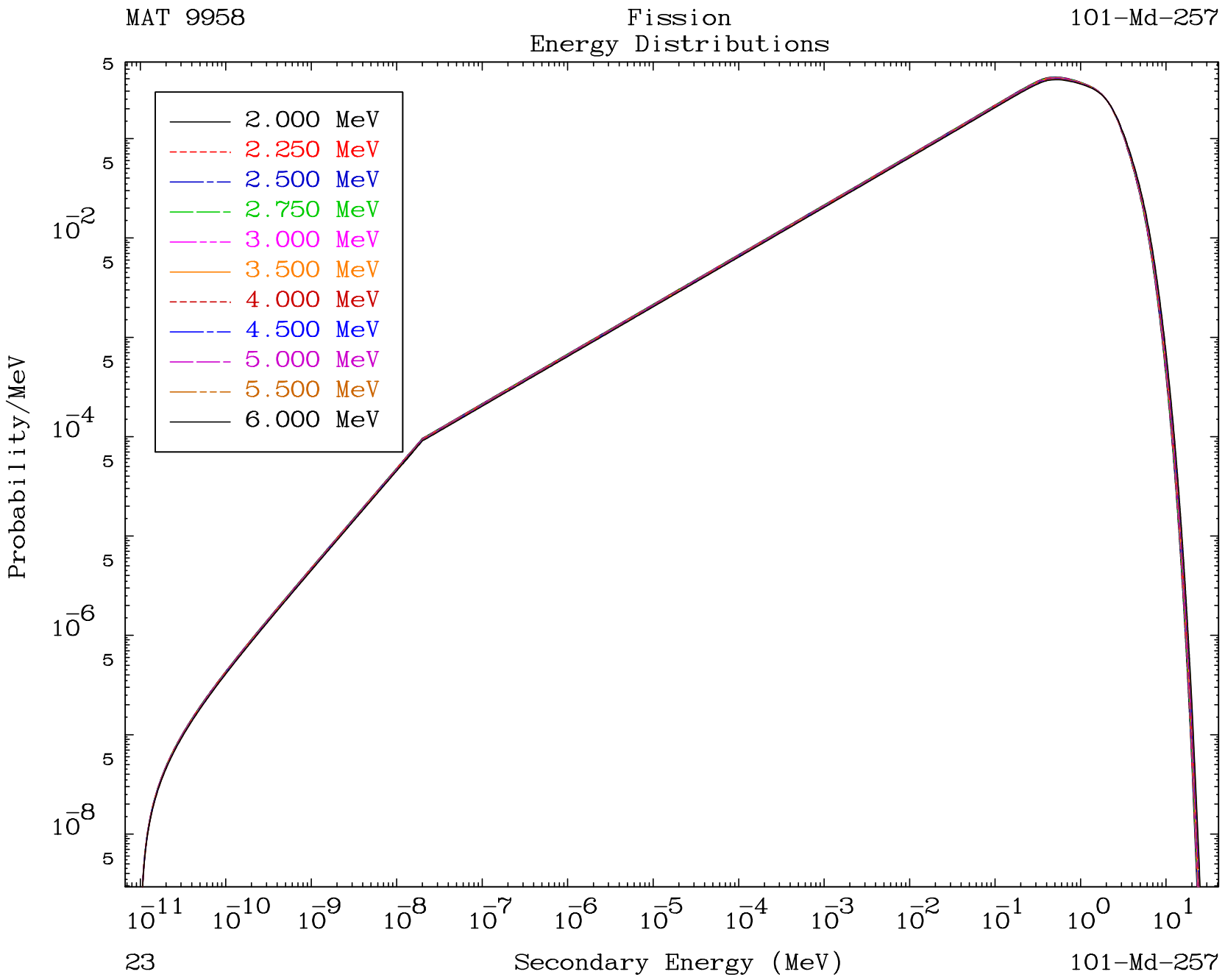

MAT 9958
Elastic
Legendre Coefficients
101-Md-257
Legendre (CM)
0.0
0.2
0.4
0.6
0.8
0
10
20
30
18
Incident Energy (MeV)
101-Md-257
P1
P2
P3
P4
P5
P6
P7
P8
P9
P10

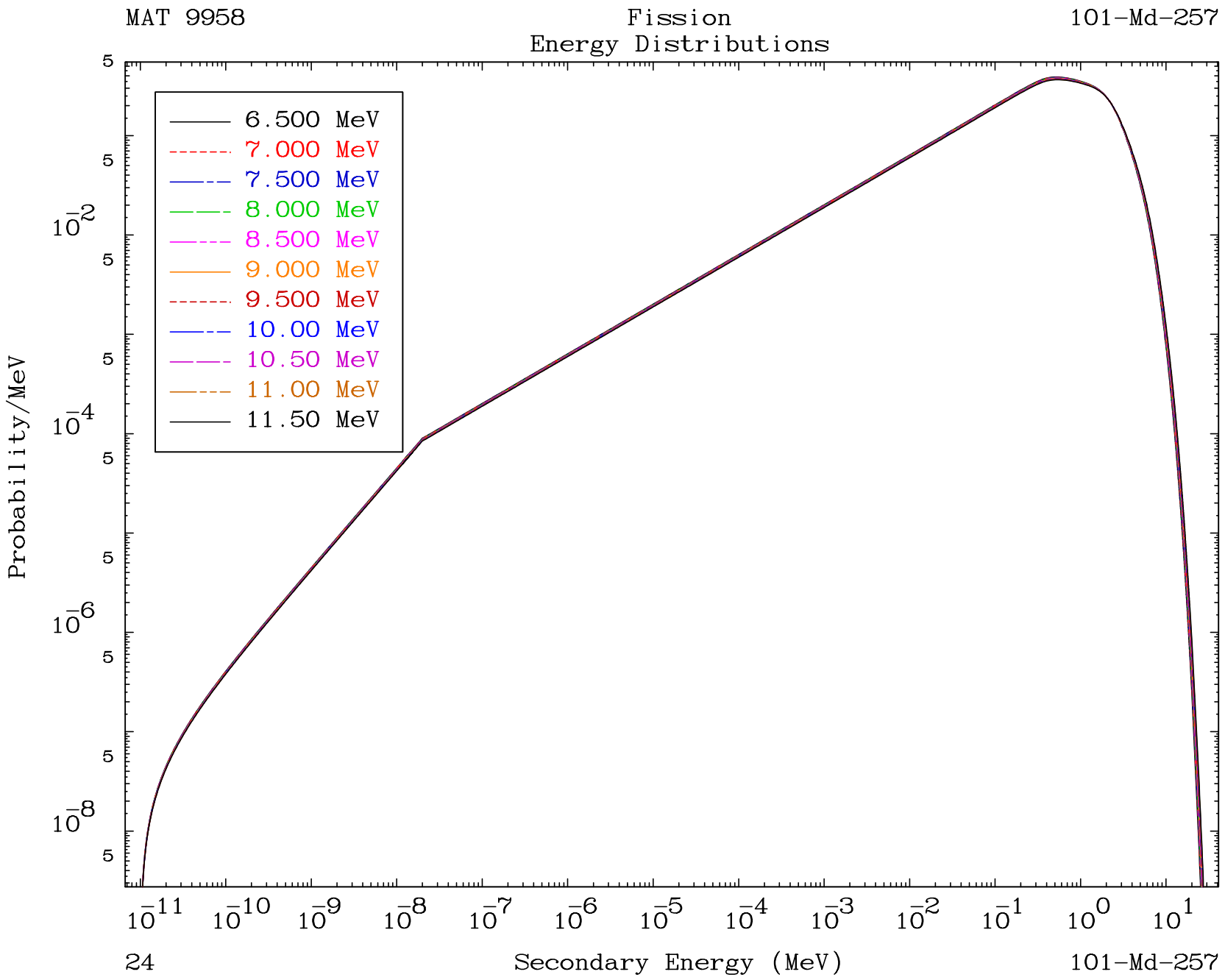

MAT 9958
Elastic
Legendre Coefficients
101-Md-257
P11
P12
P13
P14
P15
P16
P17
P18
P19
P20
Legendre (CM)
0.0
0.1
0.2
0.3
0.4
5
10
15
20
25
30
19
Incident Energy (MeV)
101-Md-257

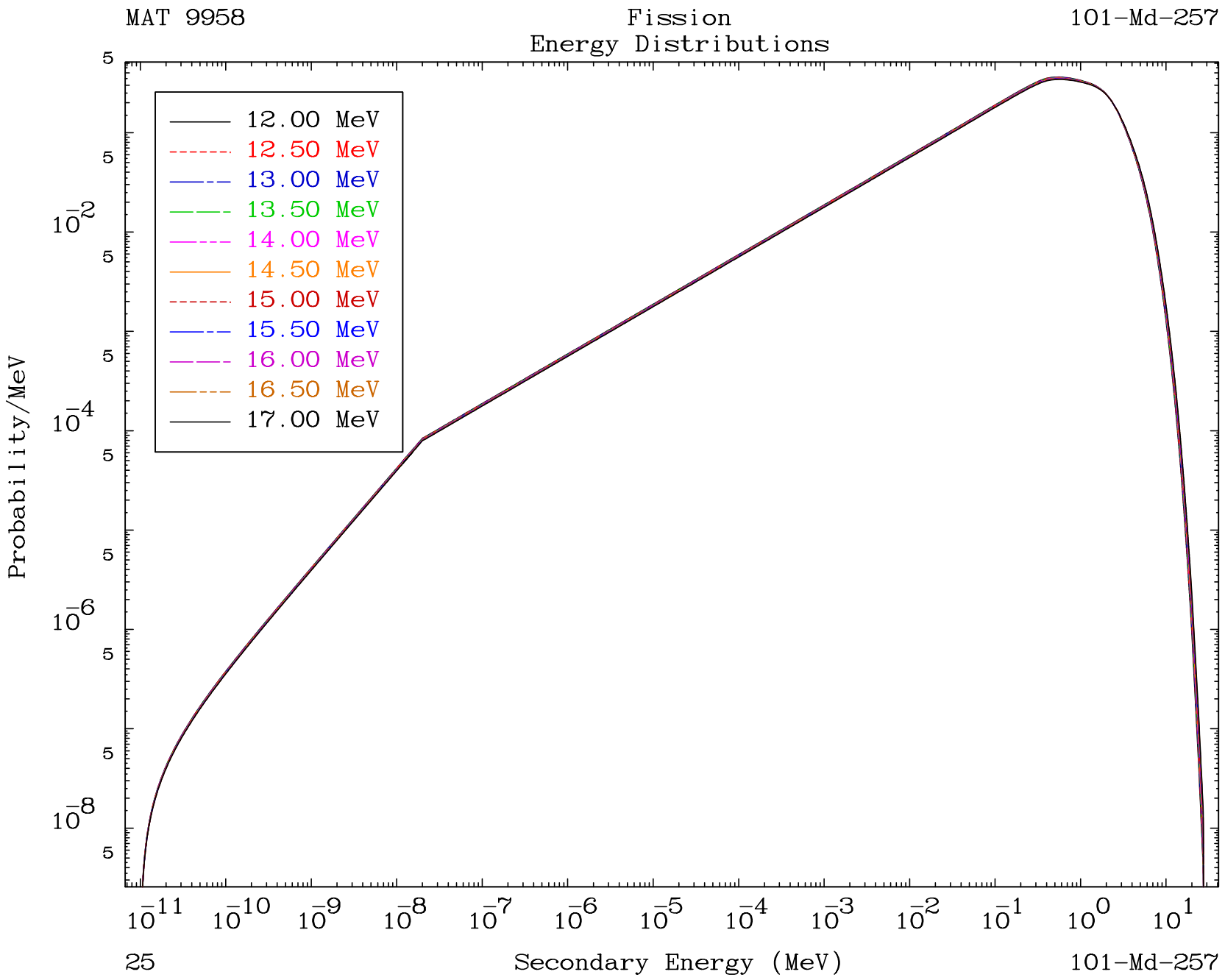

MAT 9958
Elastic
Legendre Coefficients
101-Md-257
P21
P22
P23
P24
P25
P26
P27
P28
P29
P30
0.025
0.020
0.015
0.010
0.005
0.000
Legendre (CM)
10
15
20
25
30
20
Incident Energy (MeV)
101-Md-257

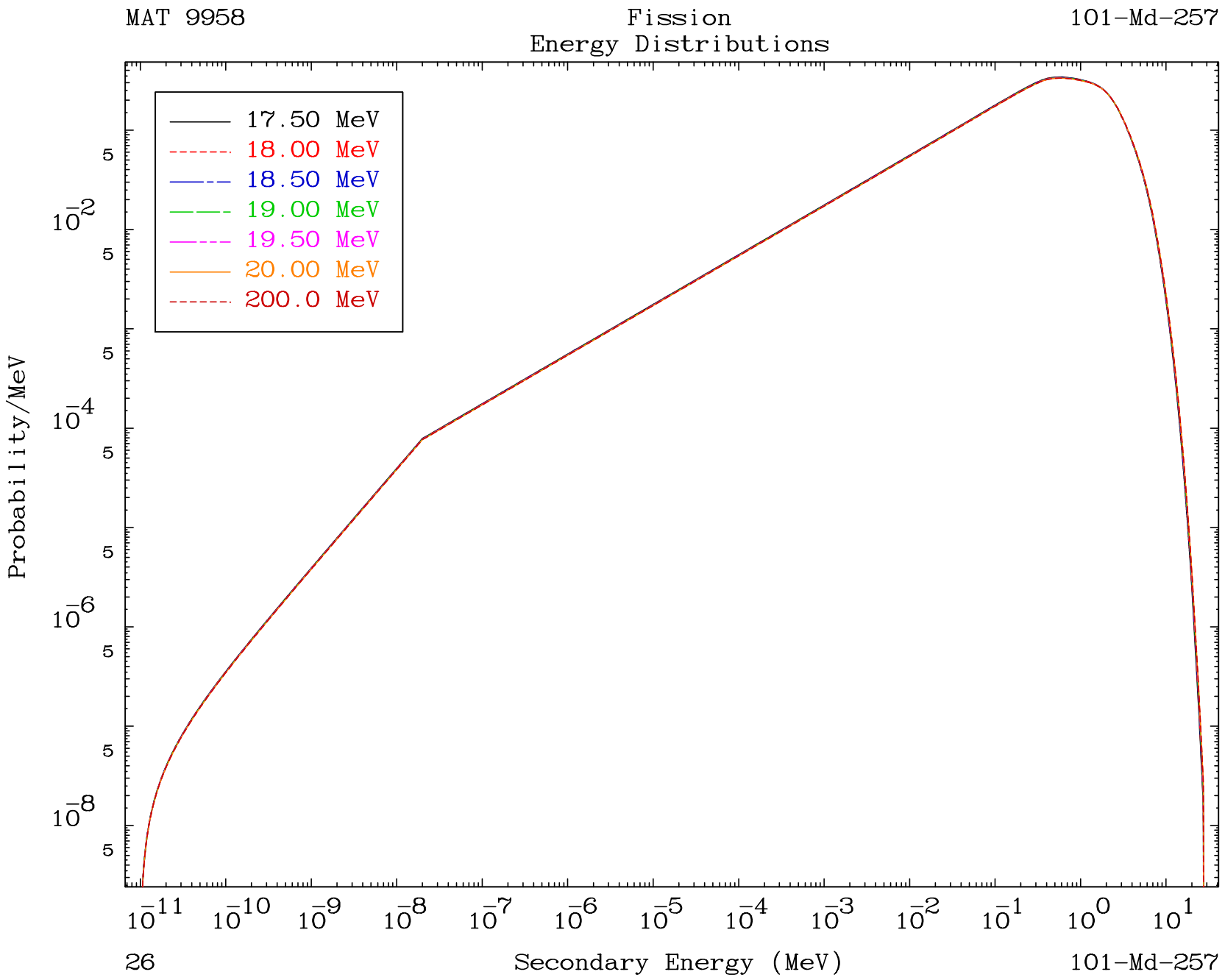

MAT 9958
Elastic
Legendre Coefficients
101-Md-257
X10 −6
3
2
1
0
Legendre (CM)
P31
P32
P33
P34
P35
P36
P37
P38
P39
P40
15
20
25
30
21
Incident Energy (MeV)
101-Md-257

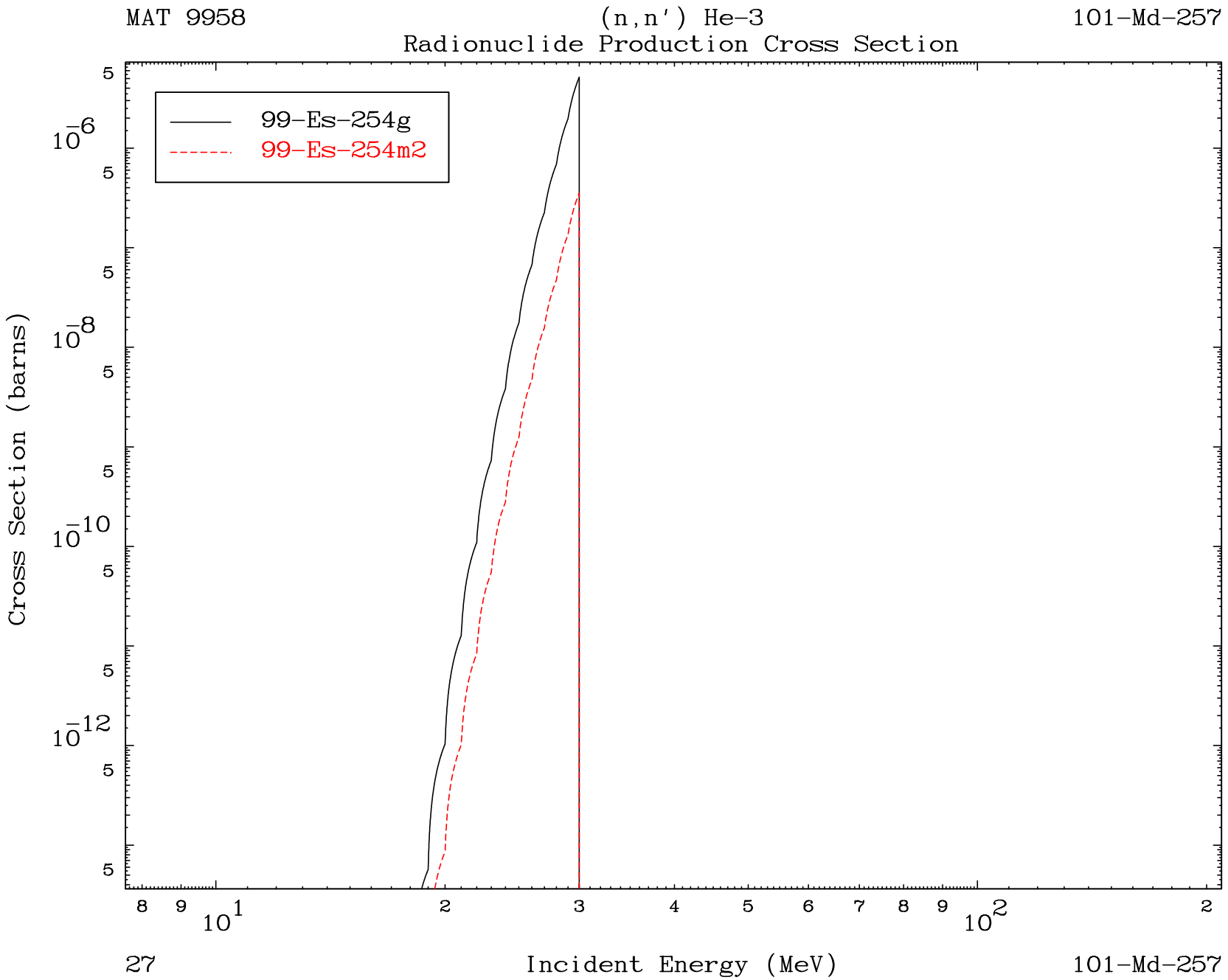

MAT 9958
Fission
Energy Distributions
101-Md-257
0.000 meV
500.0 eV
1.000 keV
10.00 keV
100.0 keV
500.0 keV
750.0 keV
1.000 MeV
1.250 MeV
1.500 MeV
1.750 MeV
Probability/MeV
Secondary Energy (MeV)
22
101-Md-257

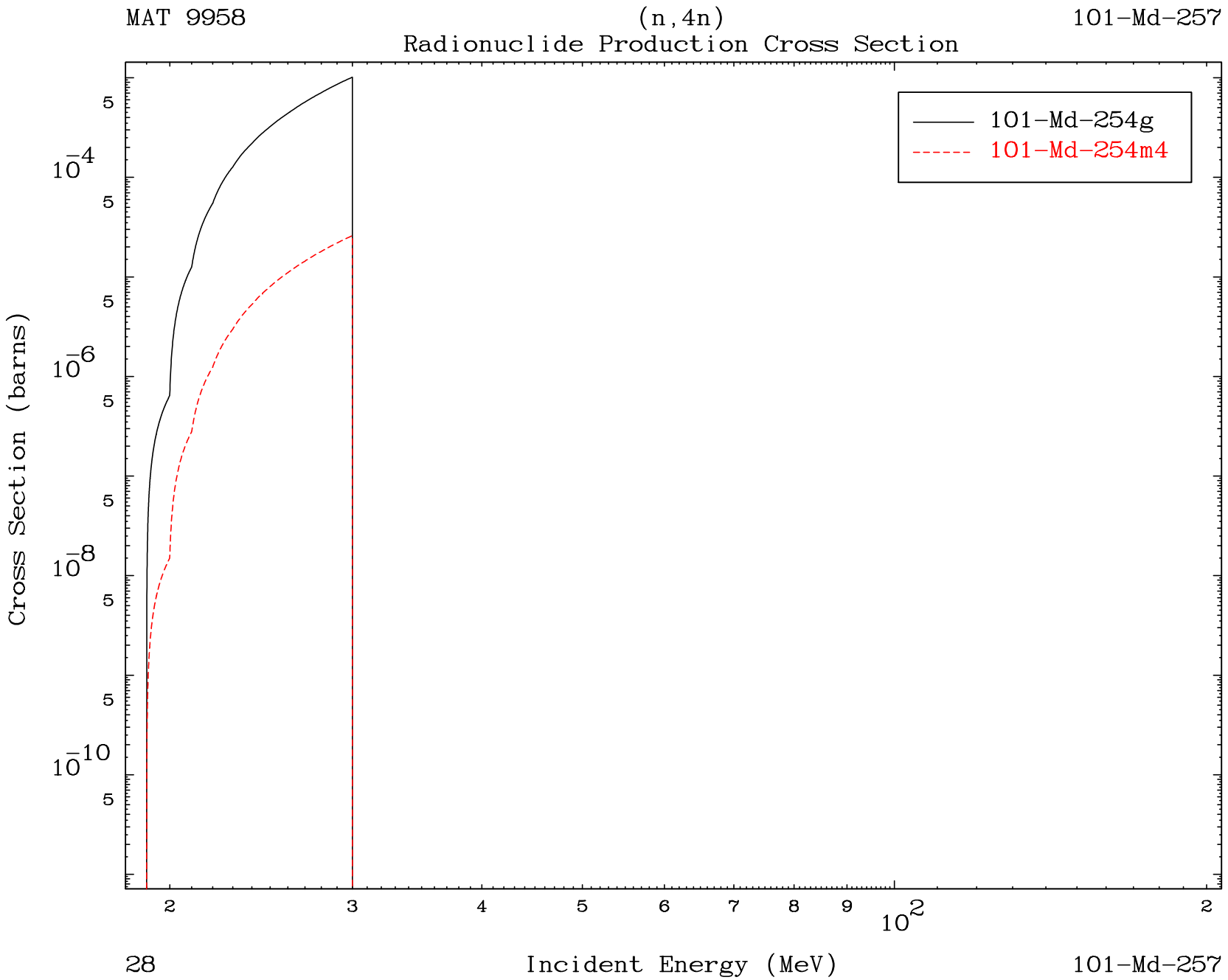

MAT 9958
Fission
Energy Distributions
101-Md-257
2.000 MeV
2.250 MeV
2.500 MeV
2.750 MeV
3.000 MeV
3.500 MeV
4.000 MeV
4.500 MeV
5.000 MeV
5.500 MeV
6.000 MeV
Probability/MeV
Secondary Energy (MeV)
23
101-Md-257

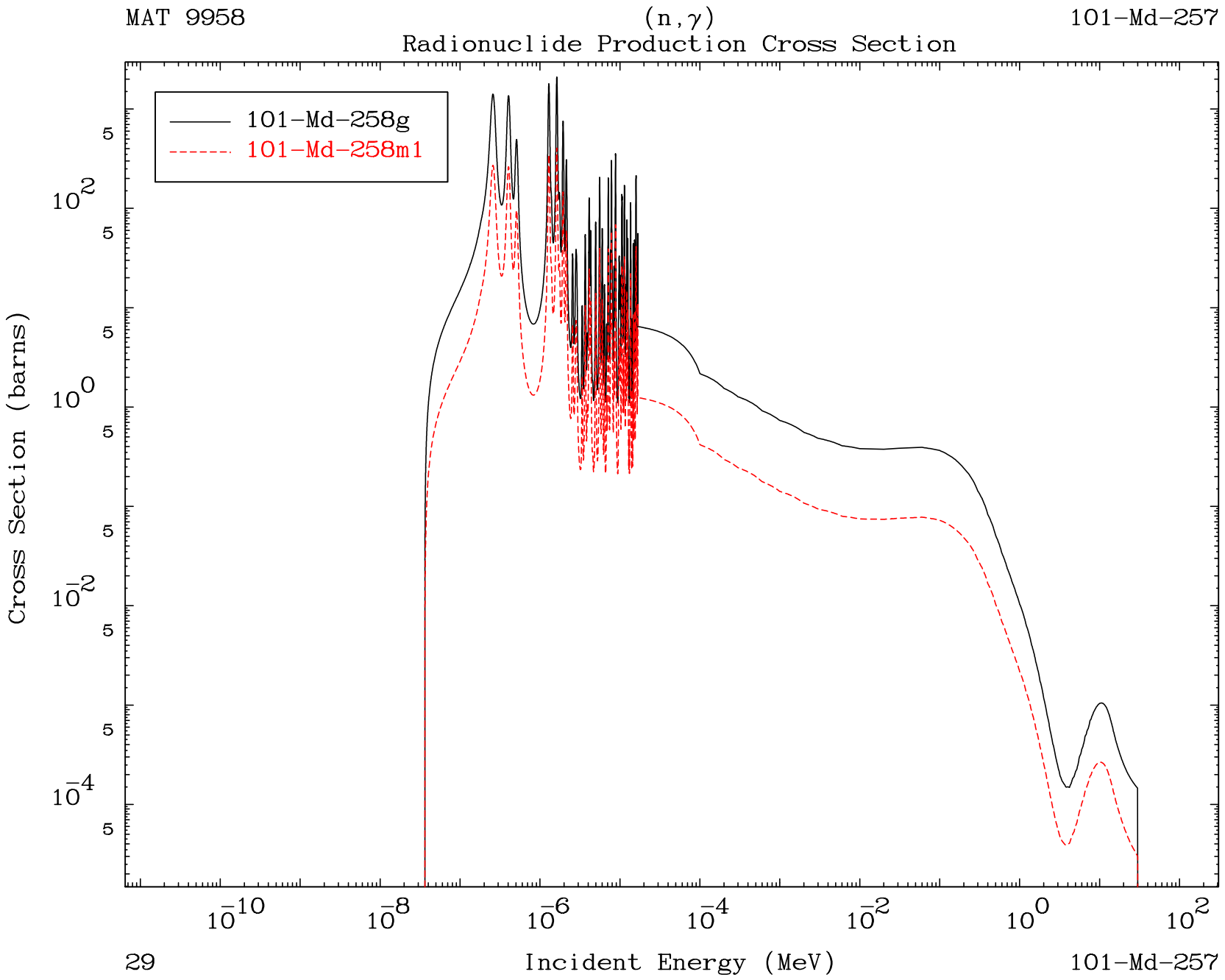

MAT 9958
Fission
Energy Distributions
101-Md-257
6.500 MeV
7.000 MeV
7.500 MeV
8.000 MeV
8.500 MeV
9.000 MeV
9.500 MeV
10.00 MeV
10.50 MeV
11.00 MeV
11.50 MeV
Probability/MeV
Secondary Energy (MeV)
24
101-Md-257

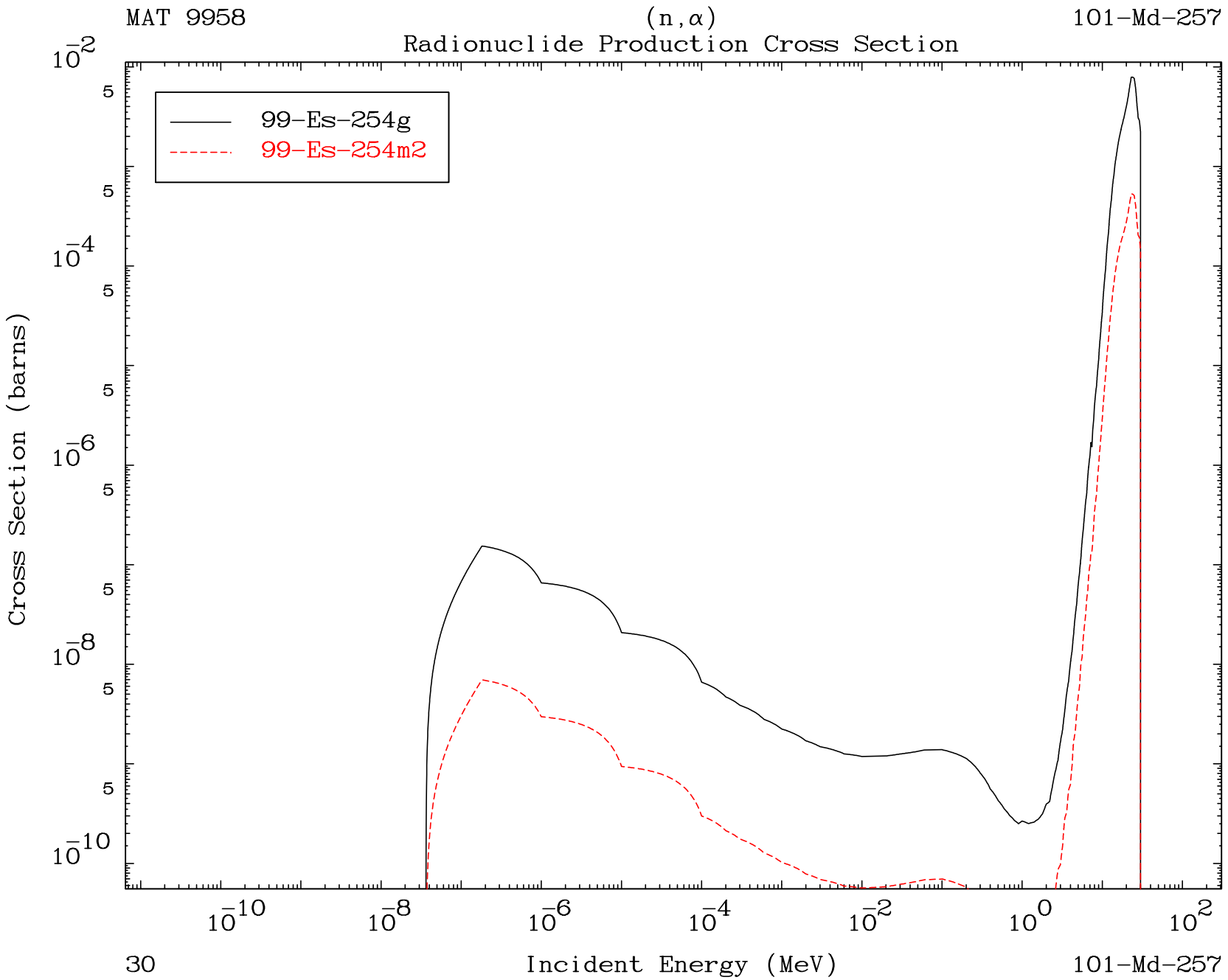

MAT 9958
Fission
Energy Distributions
101-Md-257
12.00 MeV
12.50 MeV
13.00 MeV
13.50 MeV
14.00 MeV
14.50 MeV
15.00 MeV
15.50 MeV
16.00 MeV
16.50 MeV
17.00 MeV
Probability/MeV
Secondary Energy (MeV)
25
101-Md-257


MAT 9958
Fission
Energy Distributions
101-Md-257
17.50 MeV
18.00 MeV
18.50 MeV
19.00 MeV
19.50 MeV
20.00 MeV
200.0 MeV
Probability/MeV
Secondary Energy (MeV)
26
101-Md-257


MAT 9958
(n,n') He-3
101-Md-257
Radionuclide Production Cross Section
99-Es-254g
99-Es-254m2
Cross Section (barns)
Incident Energy (MeV)
27
101-Md-257


MAT 9958
(n,4n)
101-Md-257
Radionuclide Production Cross Section
101-Md-254g
101-Md-254m4
Cross Section (barns)
10-4
10-6
10-8
10-10
2
3
4
5
6
7
8
9
10 2
28
Incident Energy (MeV)
101-Md-257


MAT 9958
(n,γ)
101-Md-257
Radionuclide Production Cross Section
101-Md-258g
101-Md-258m1
Cross Section (barns)
10^2
10^0
10^-2
10^-4
10^-10
10^-8
10^-6
10^-4
10^-2
10^0
10^2
29
Incident Energy (MeV)
101-Md-257


MAT 9958
(n,α)
101-Md-257
Radionuclide Production Cross Section
99-Es-254g
99-Es-254m2
Cross Section (barns)
Incident Energy (MeV)
30
101-Md-257