

Program EVALPLOT  
(Version 2021-1)

by

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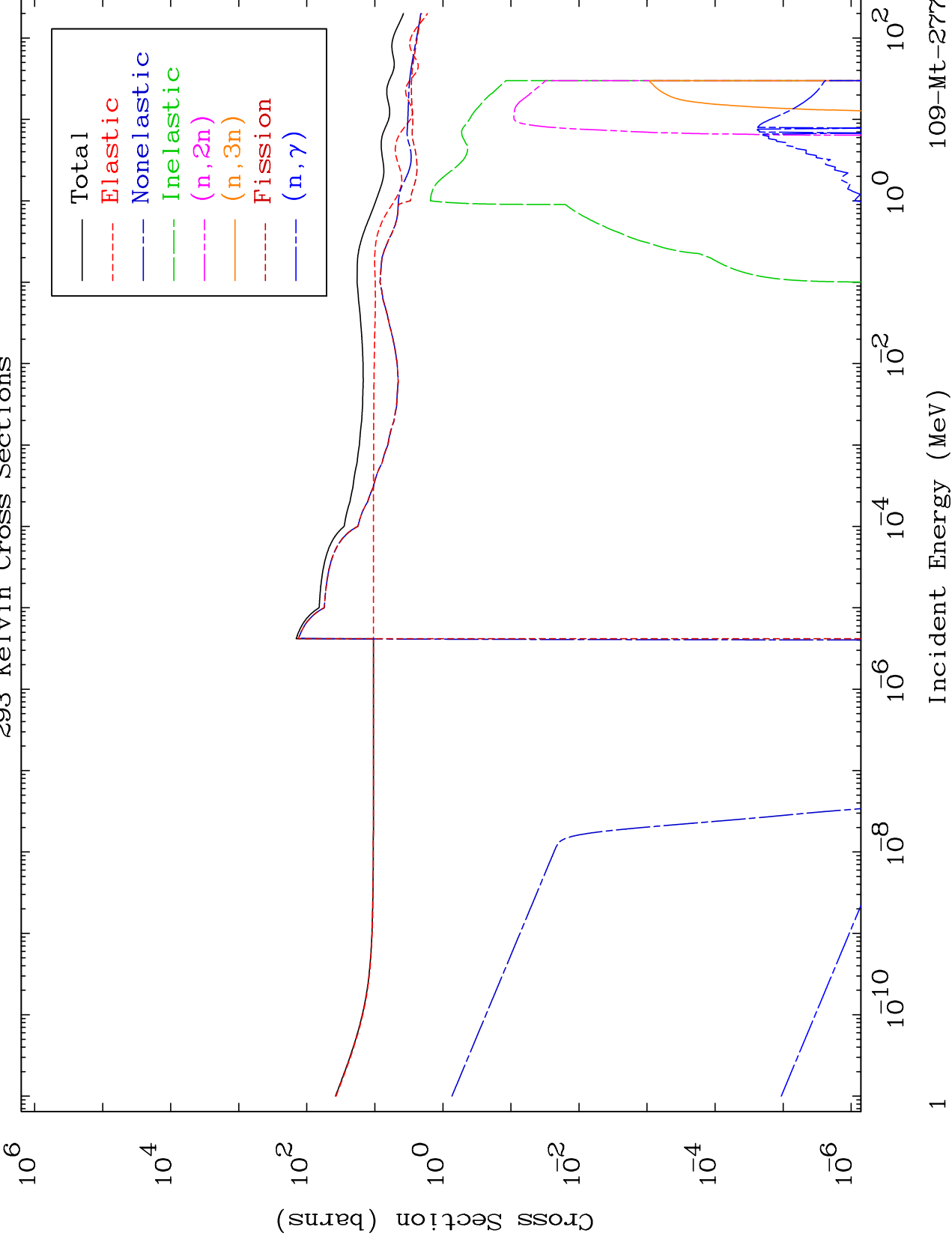
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 977

Neutron Major  
293 Kelvin Cross Sections

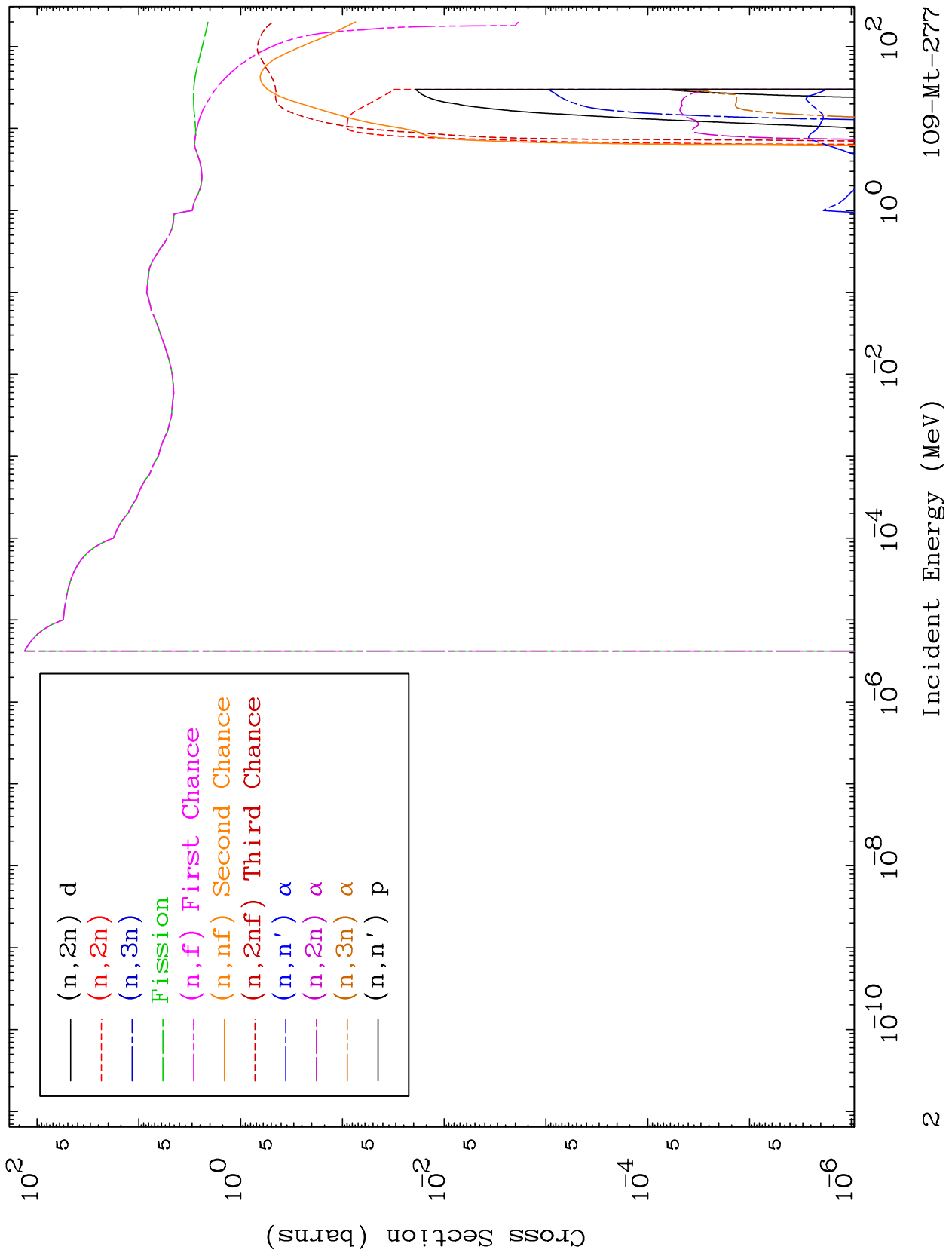
109-Mt-277



MAT 977

Neutron Absorption  
293 Kelvin Cross Sections

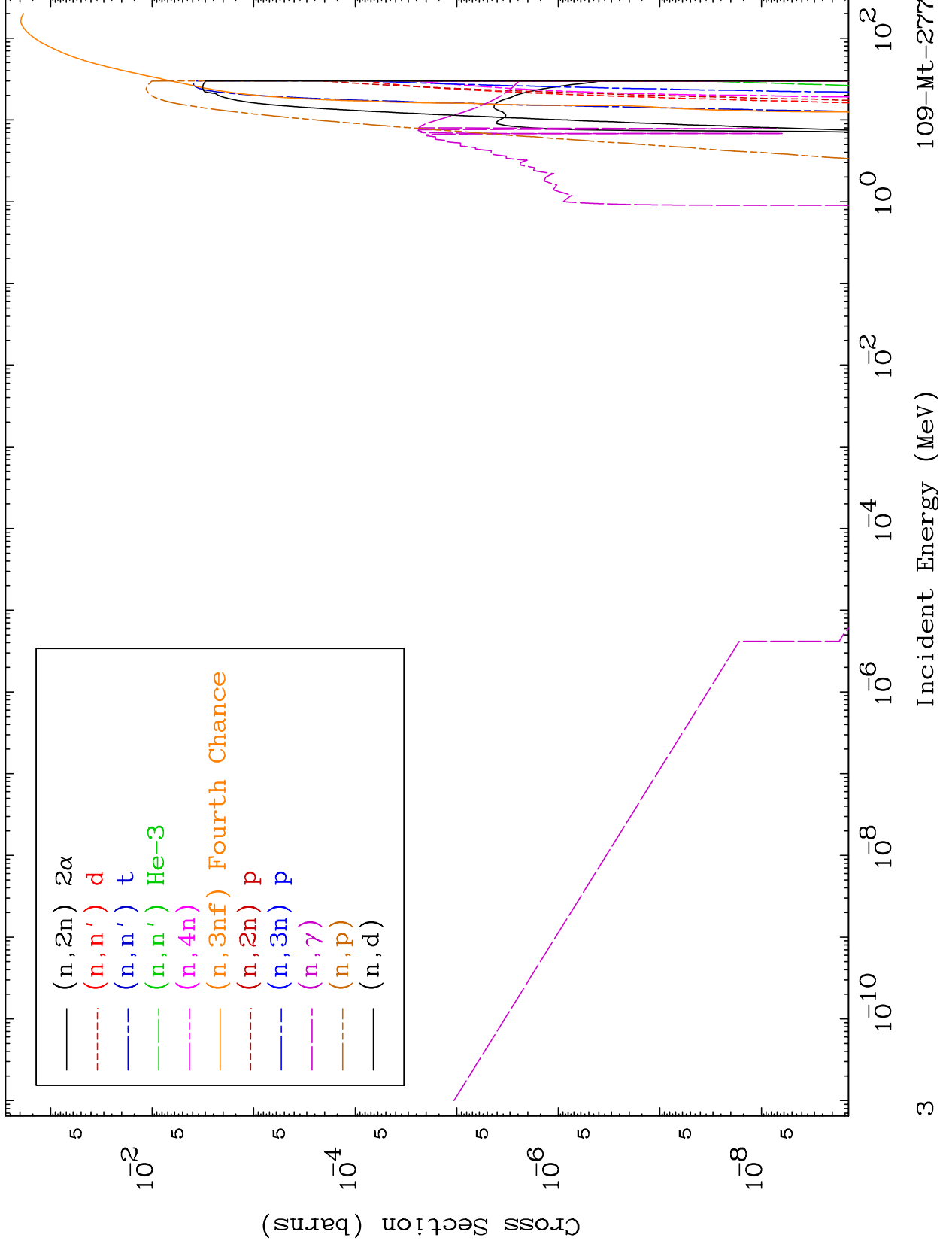
109-Mt-277

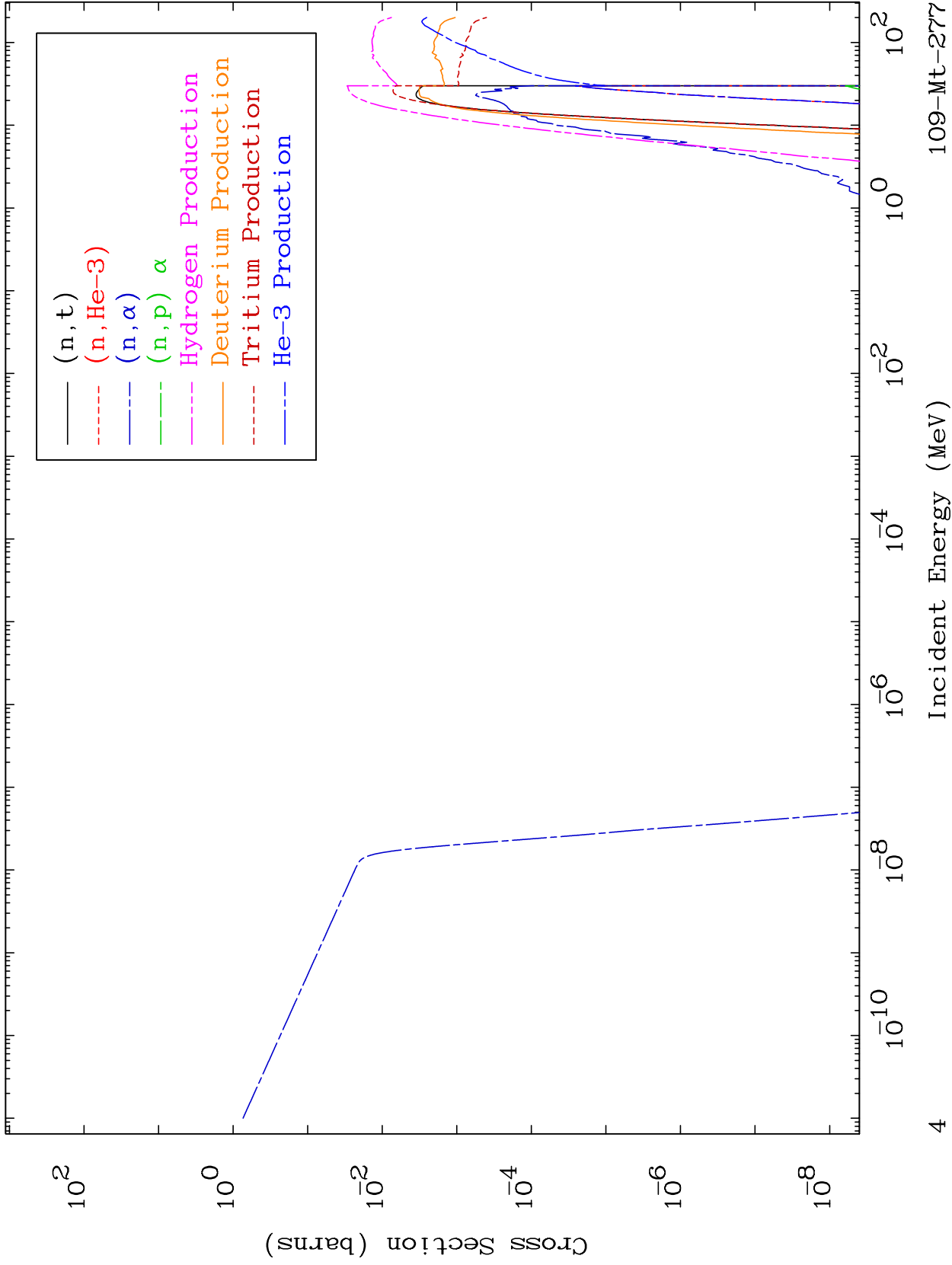


MAT 977

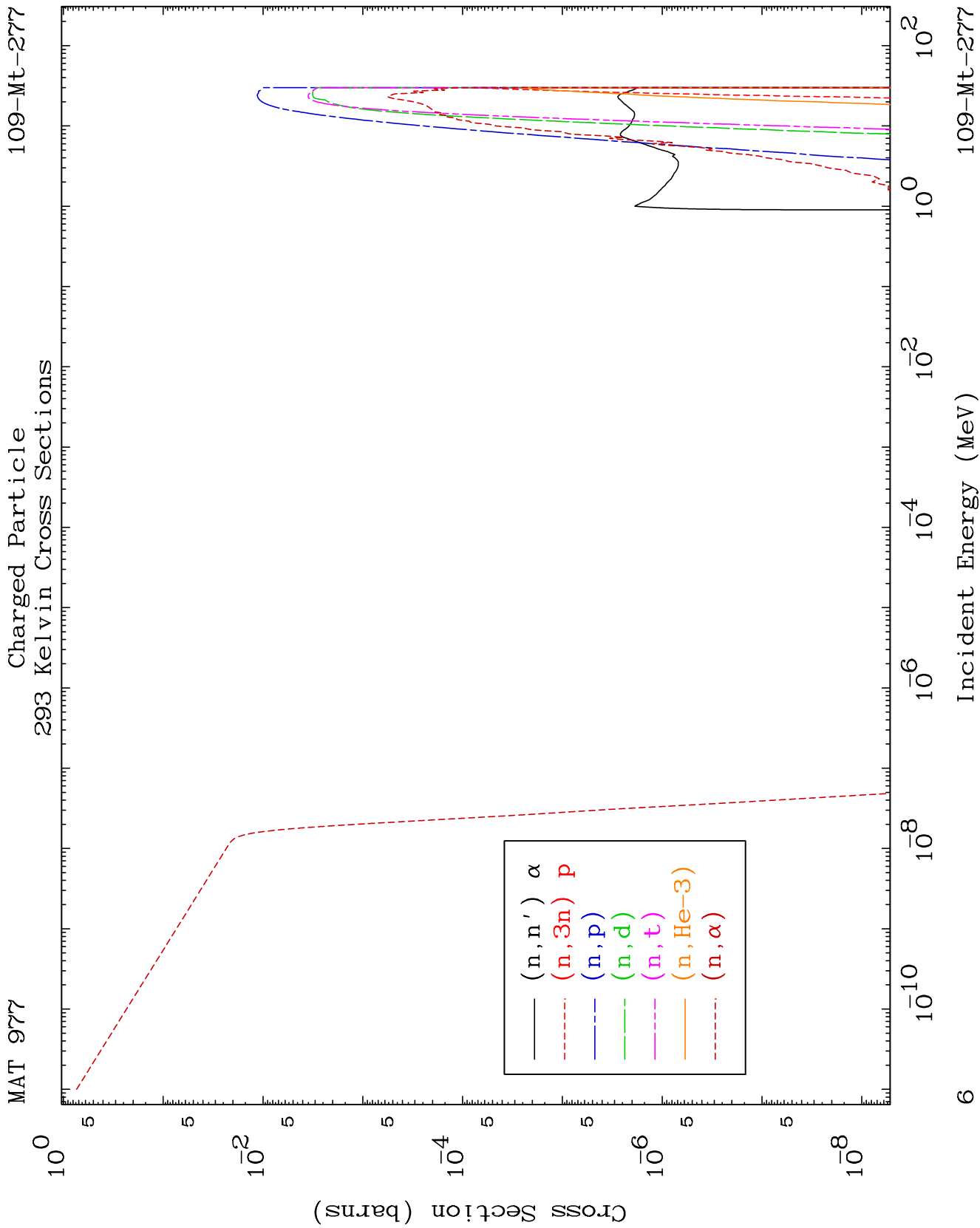
Neutron Absorption  
293 Kelvin Cross Sections

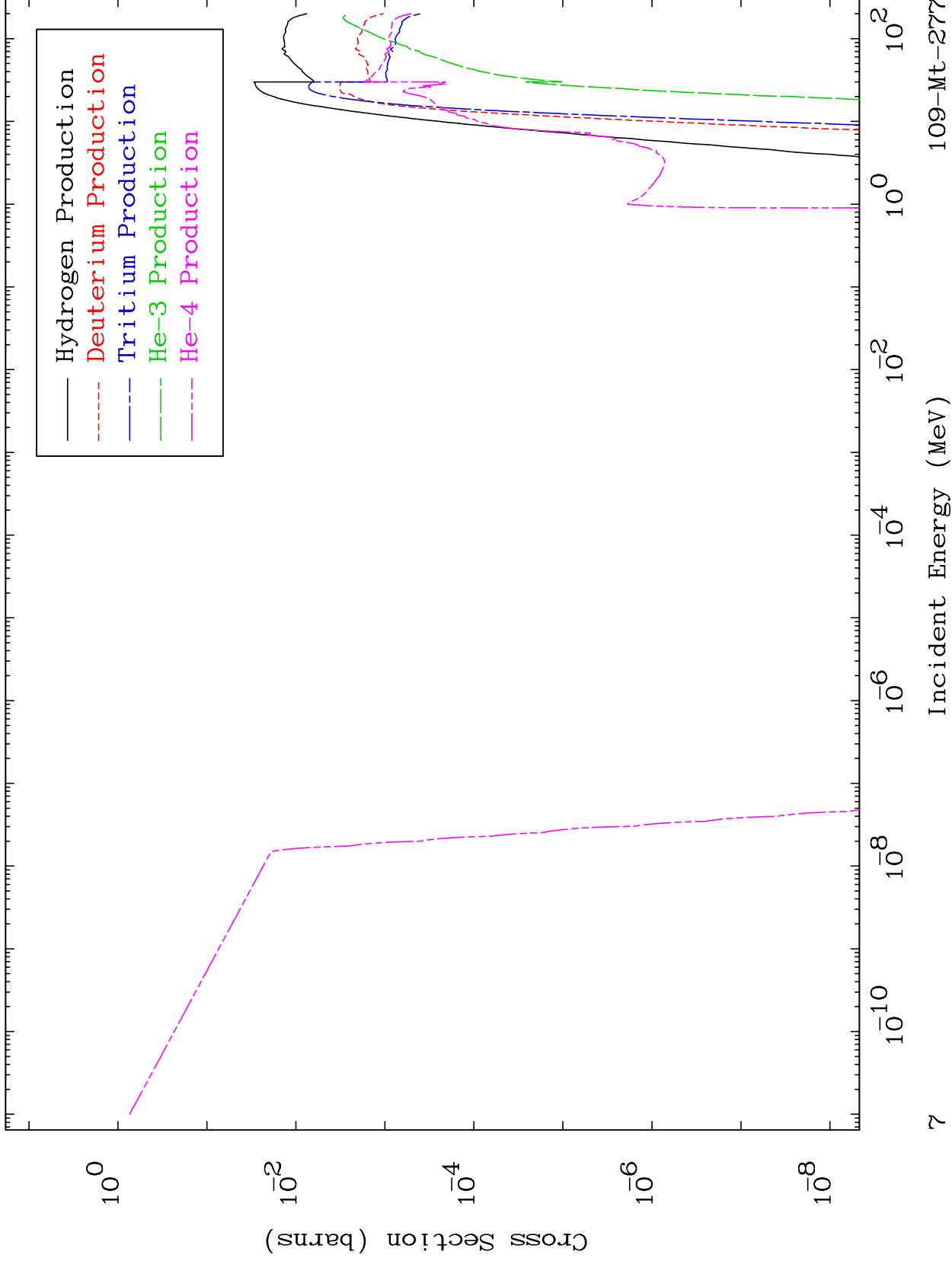
109-Mt-277









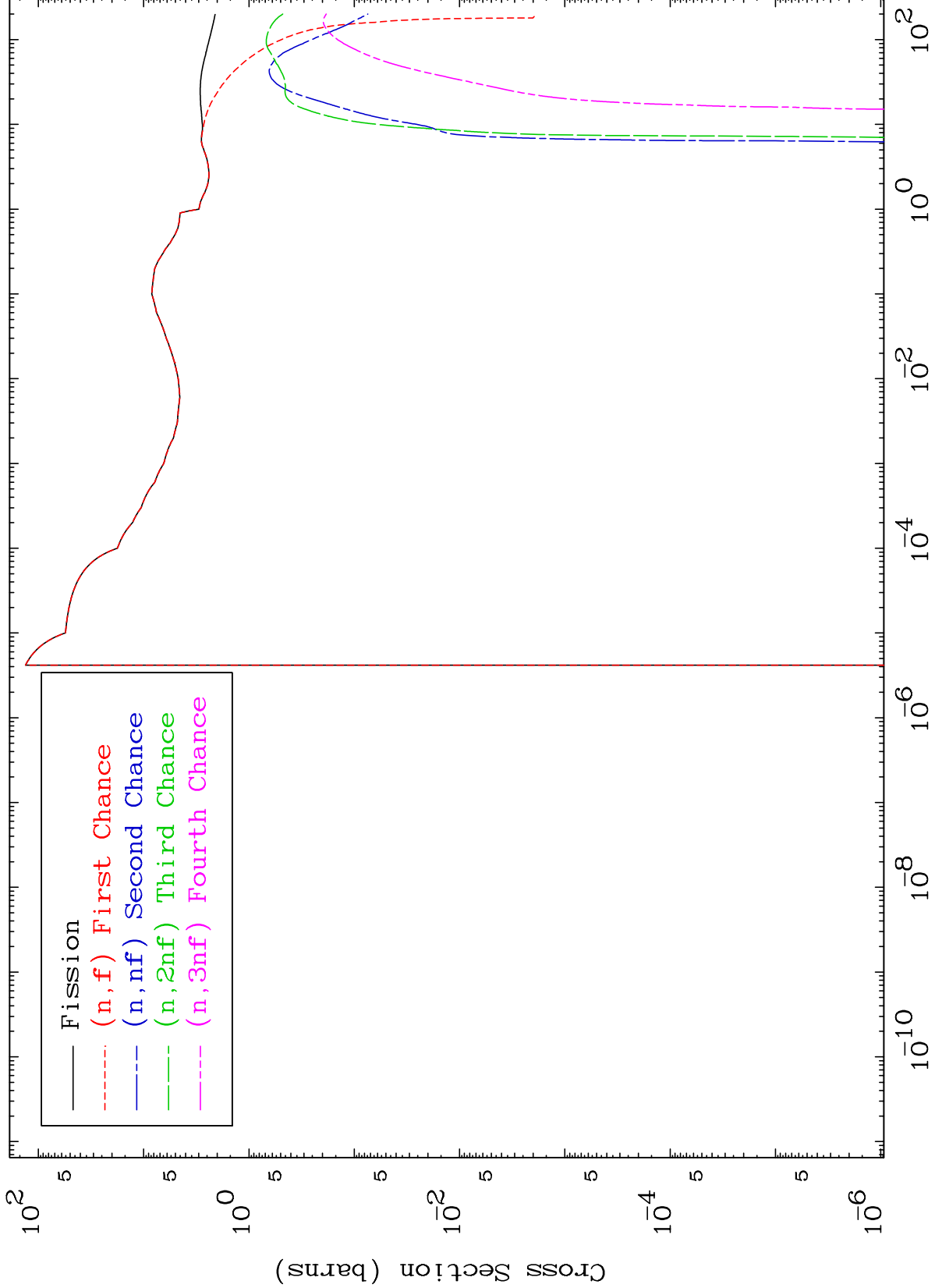




MAT 977

Fission  
293 Kelvin Cross Sections

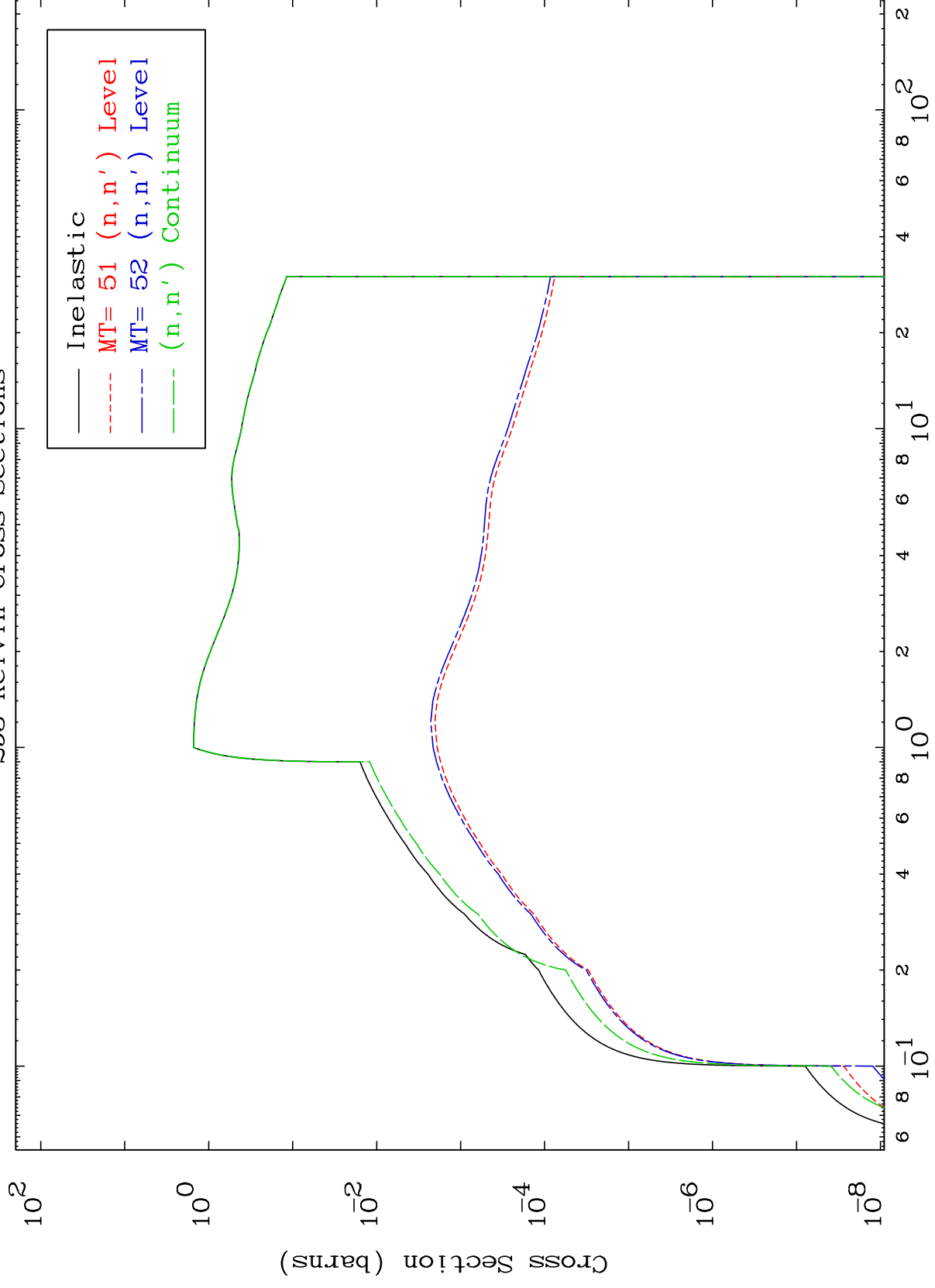
109-Mt-277



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(n,n') Levels  
293 Kelvin Cross Sections

109-Mt-277



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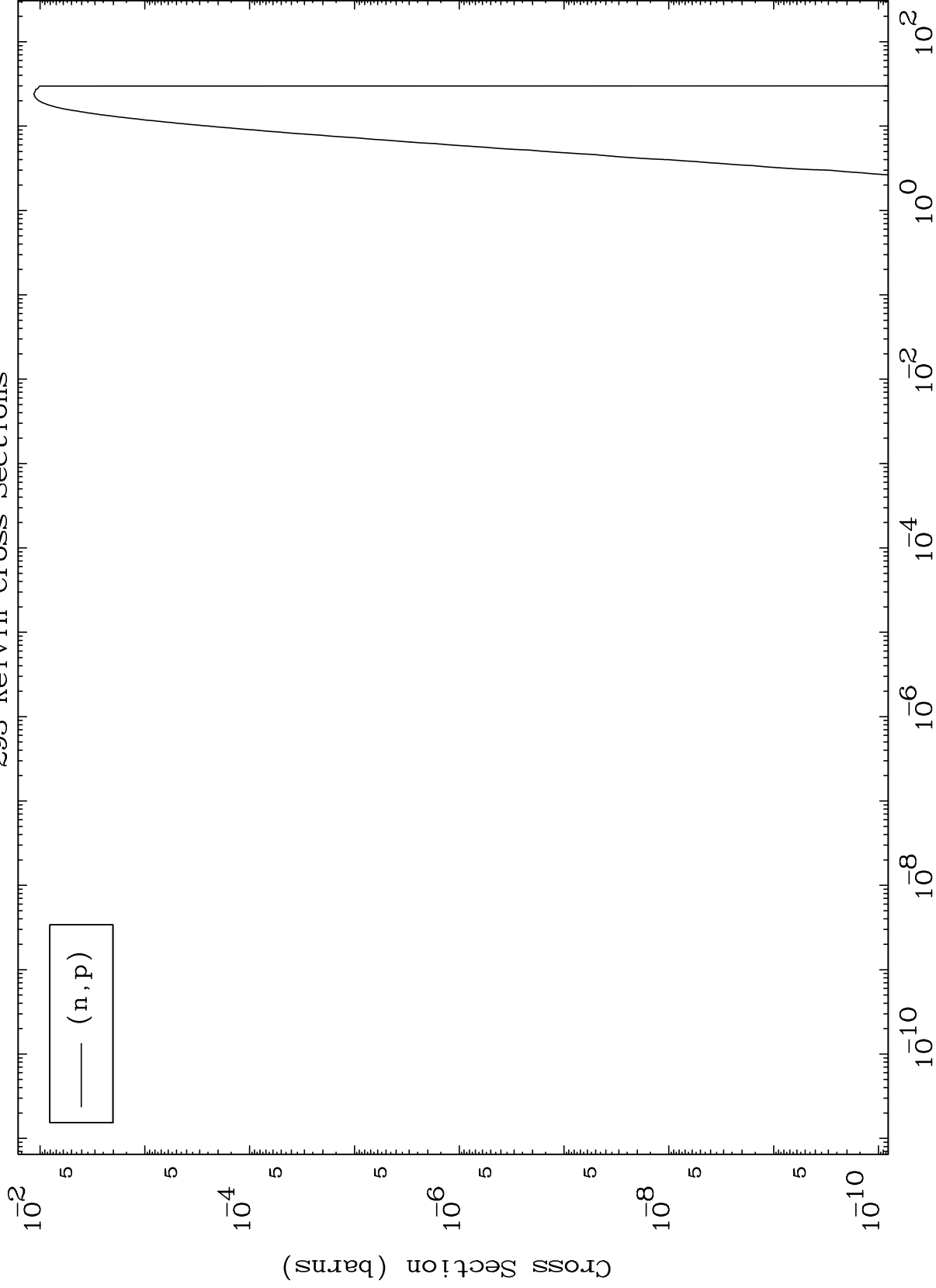
Incident Energy (MeV)

109-Mt-277

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(n,p) Levels  
293 Kelvin Cross Sections

109-Mt-277



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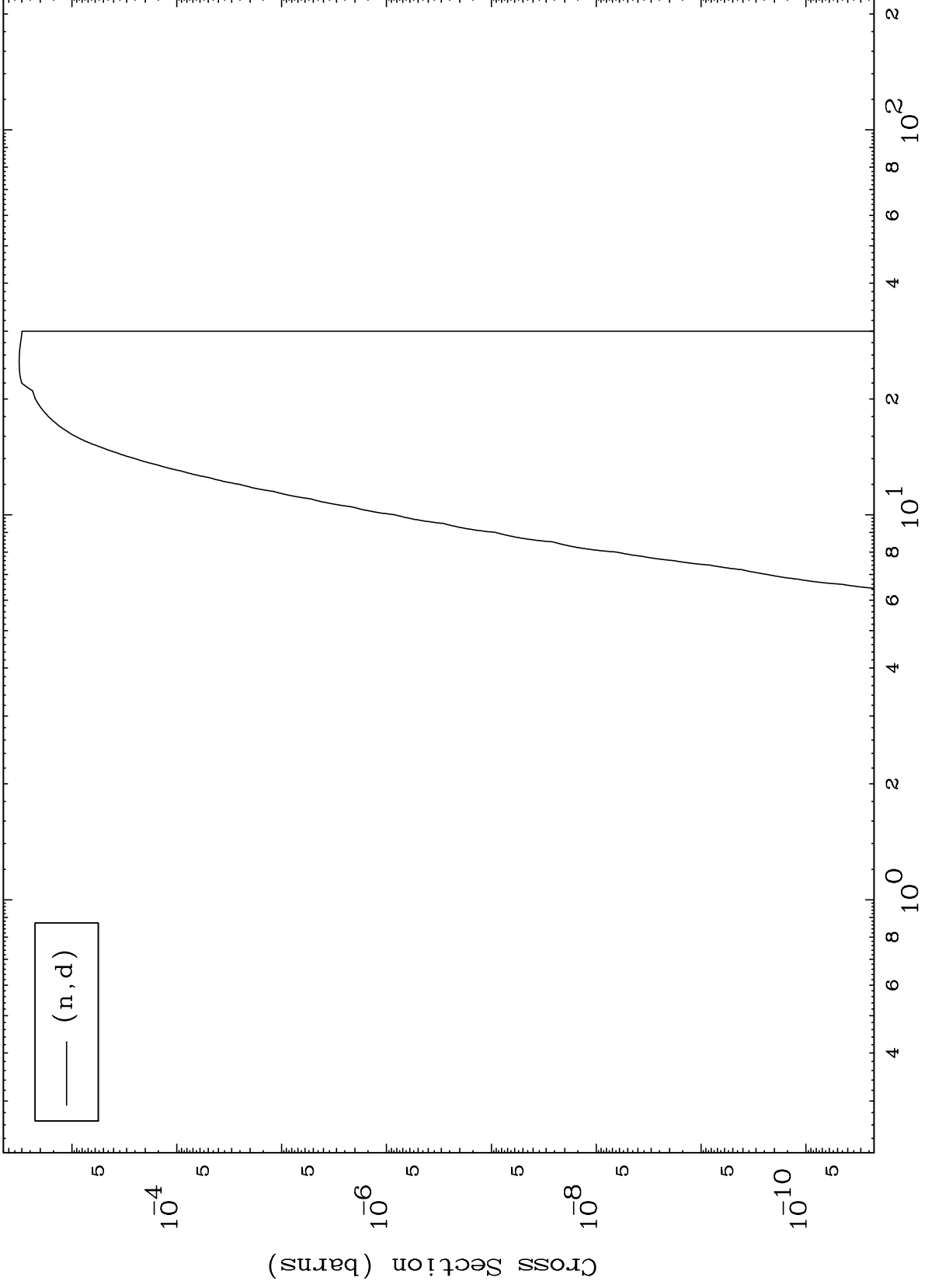
Incident Energy (MeV)

109-Mt-277

MAT 977

(n,d) Levels  
293 Kelvin Cross Sections

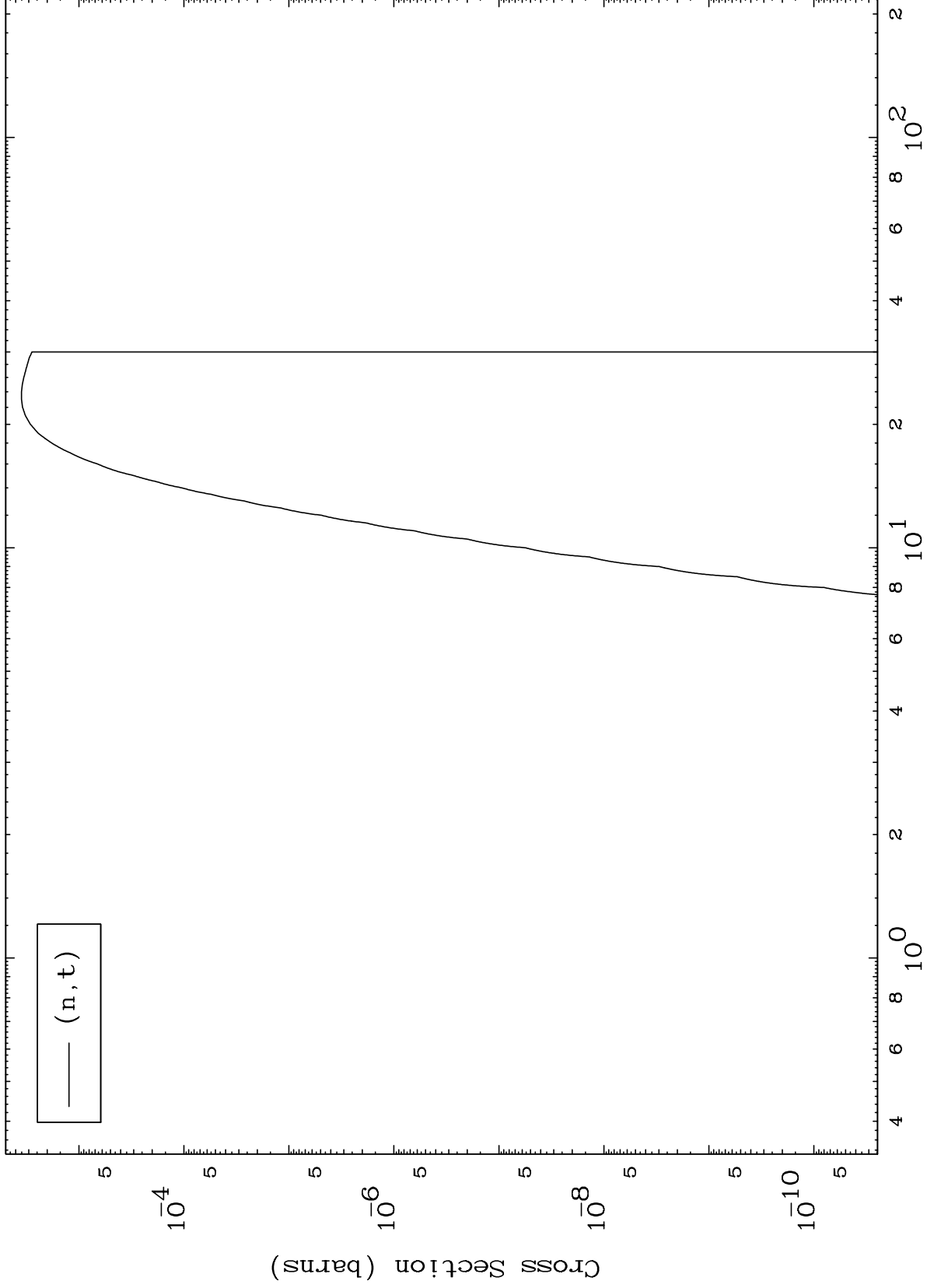
109-Mt-277



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(n,t) Levels  
293 Kelvin Cross Sections

109-Mt-277



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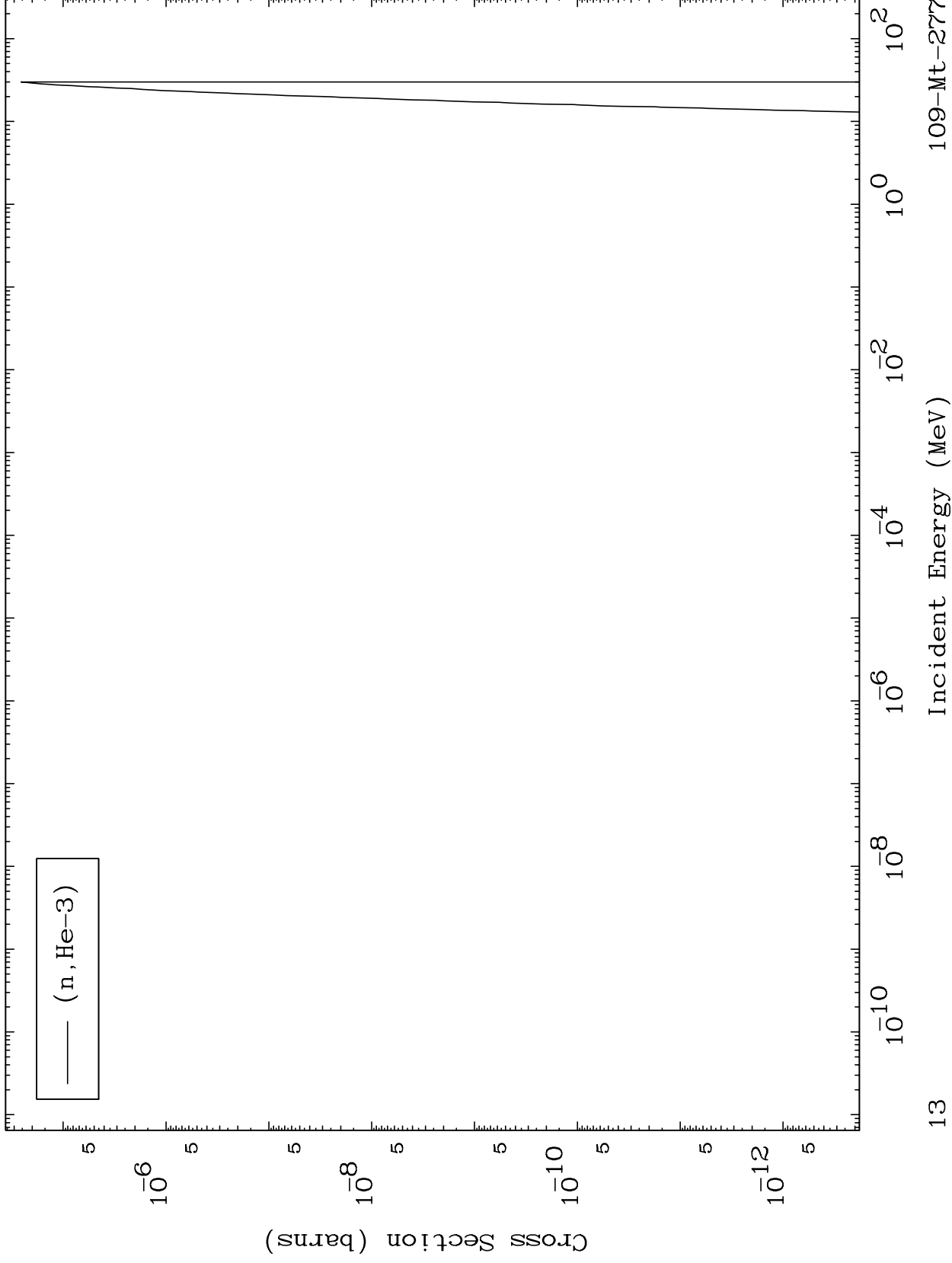
Incident Energy (MeV)

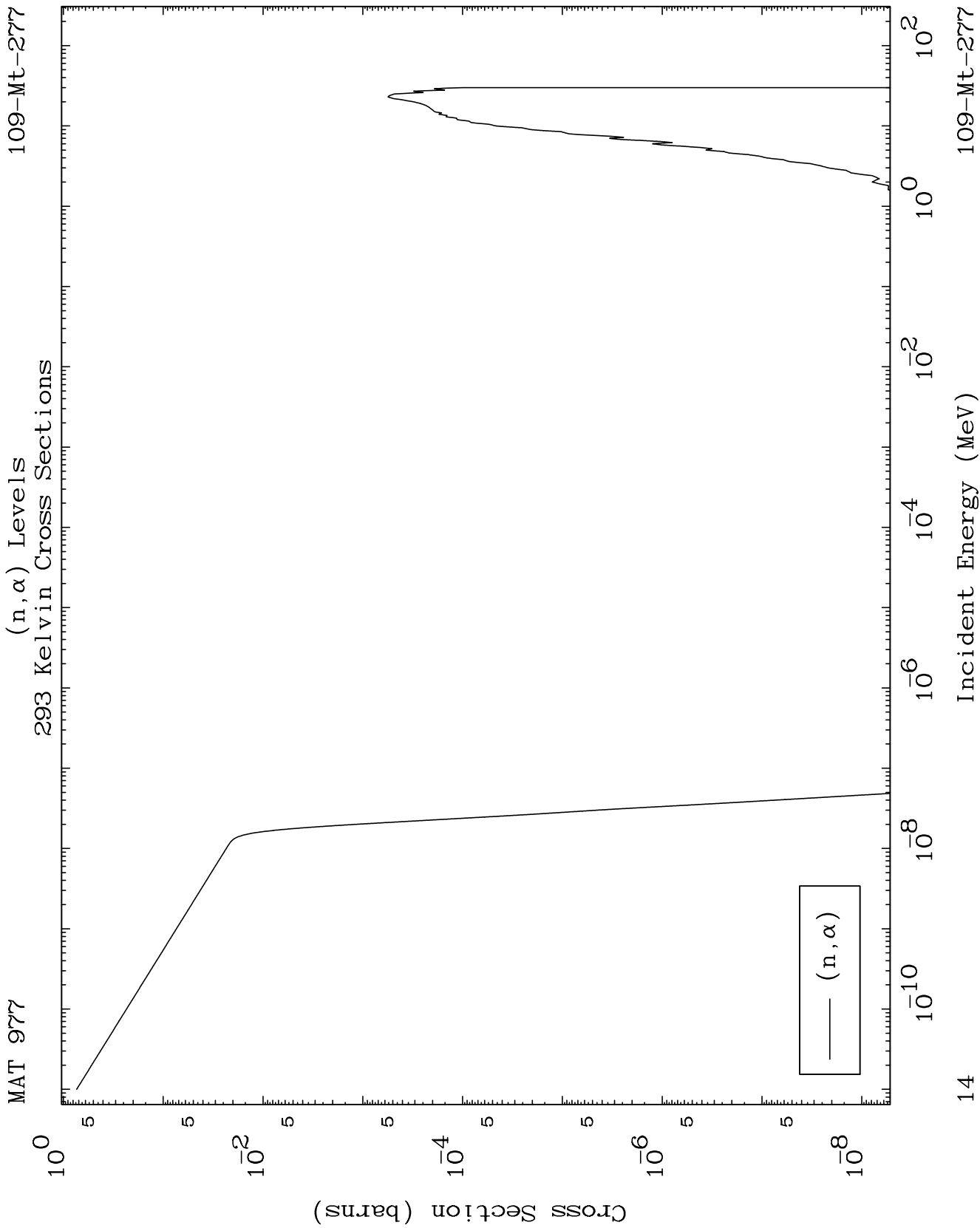
109-Mt-277

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(n,He3) Levels  
293 Kelvin Cross Sections

109-Mt-277

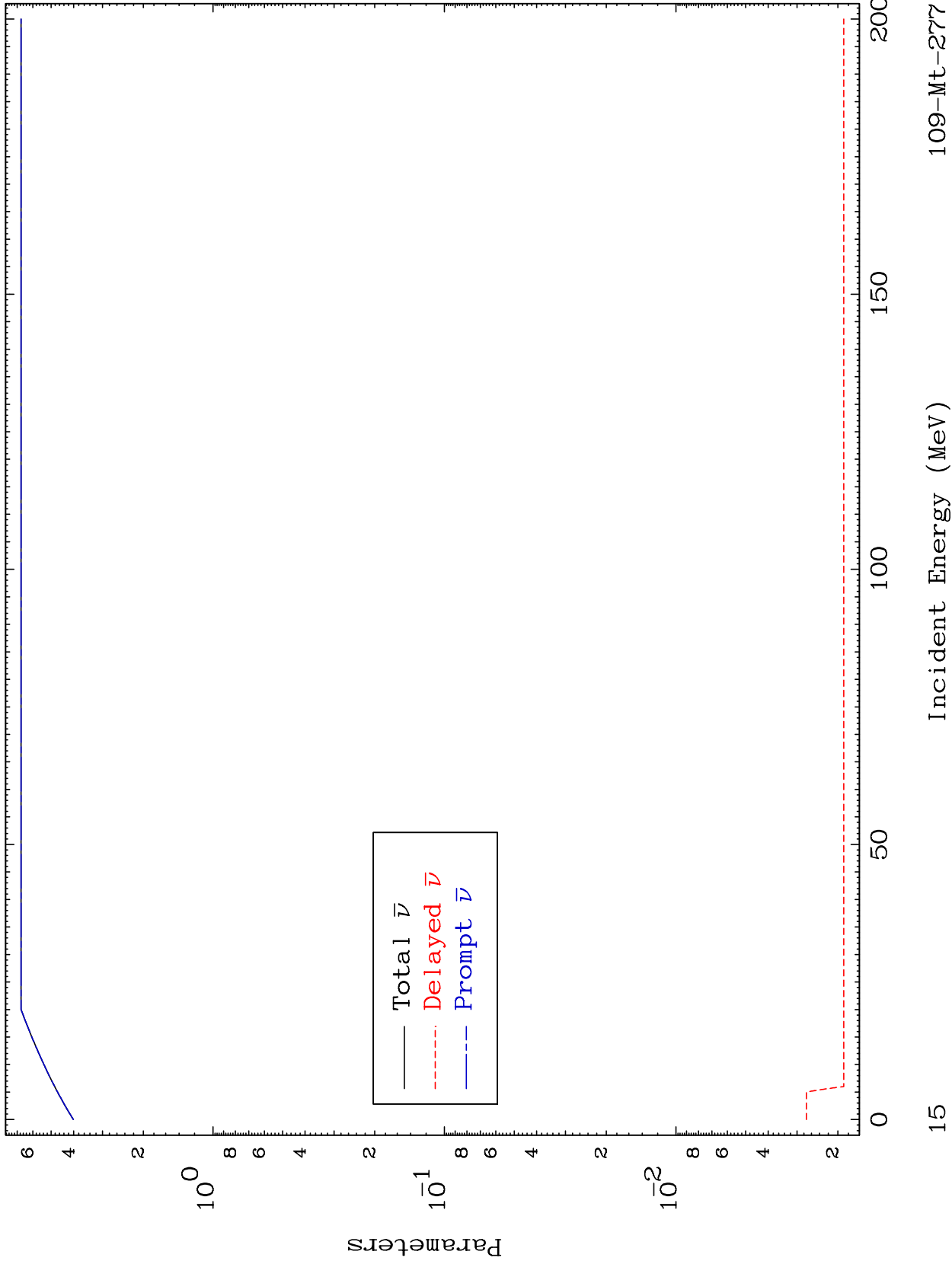




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Energy Release  
Parameters

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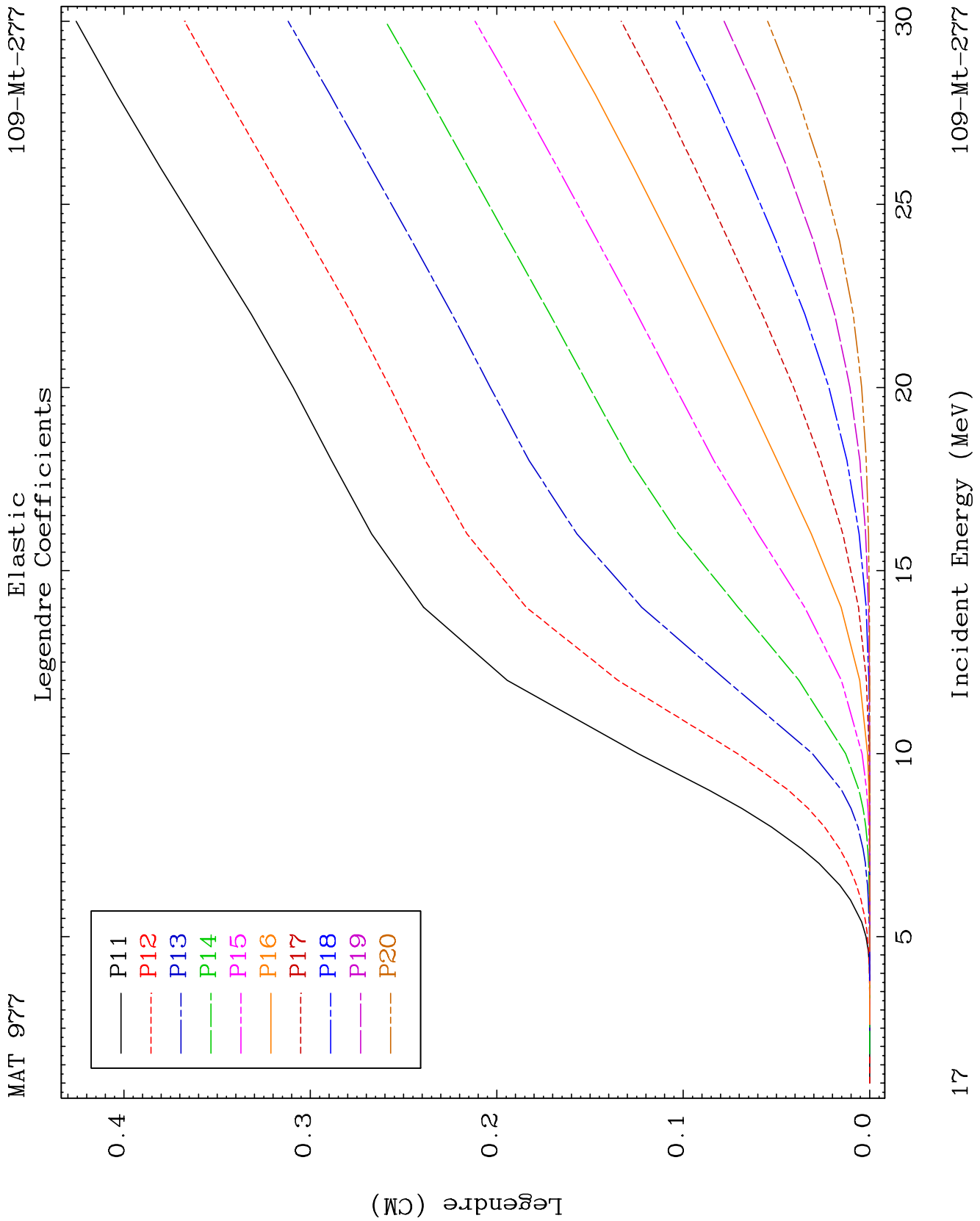
15

Incident Energy (MeV)

109-Mt-277



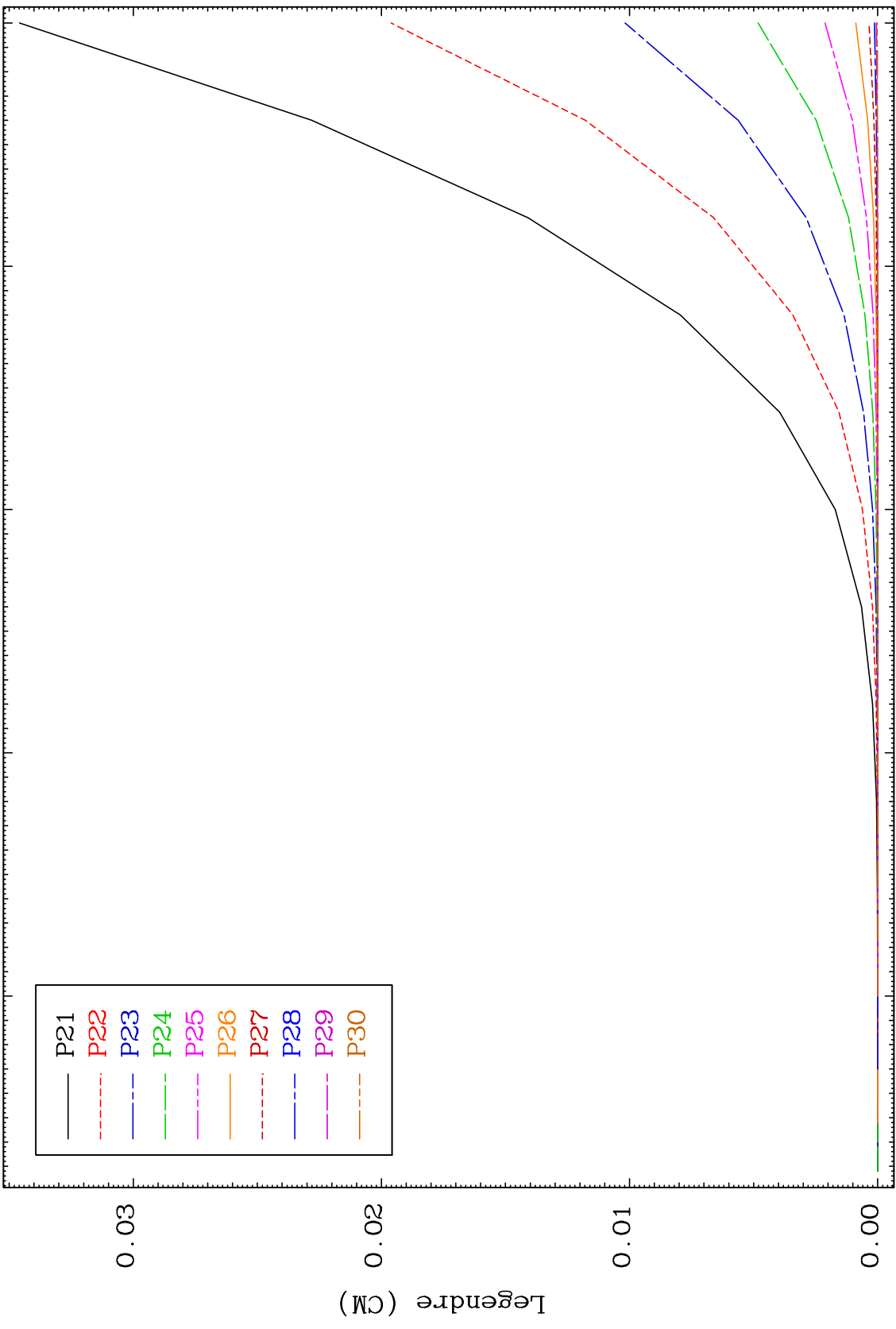




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### Elastic Legendre Coefficients

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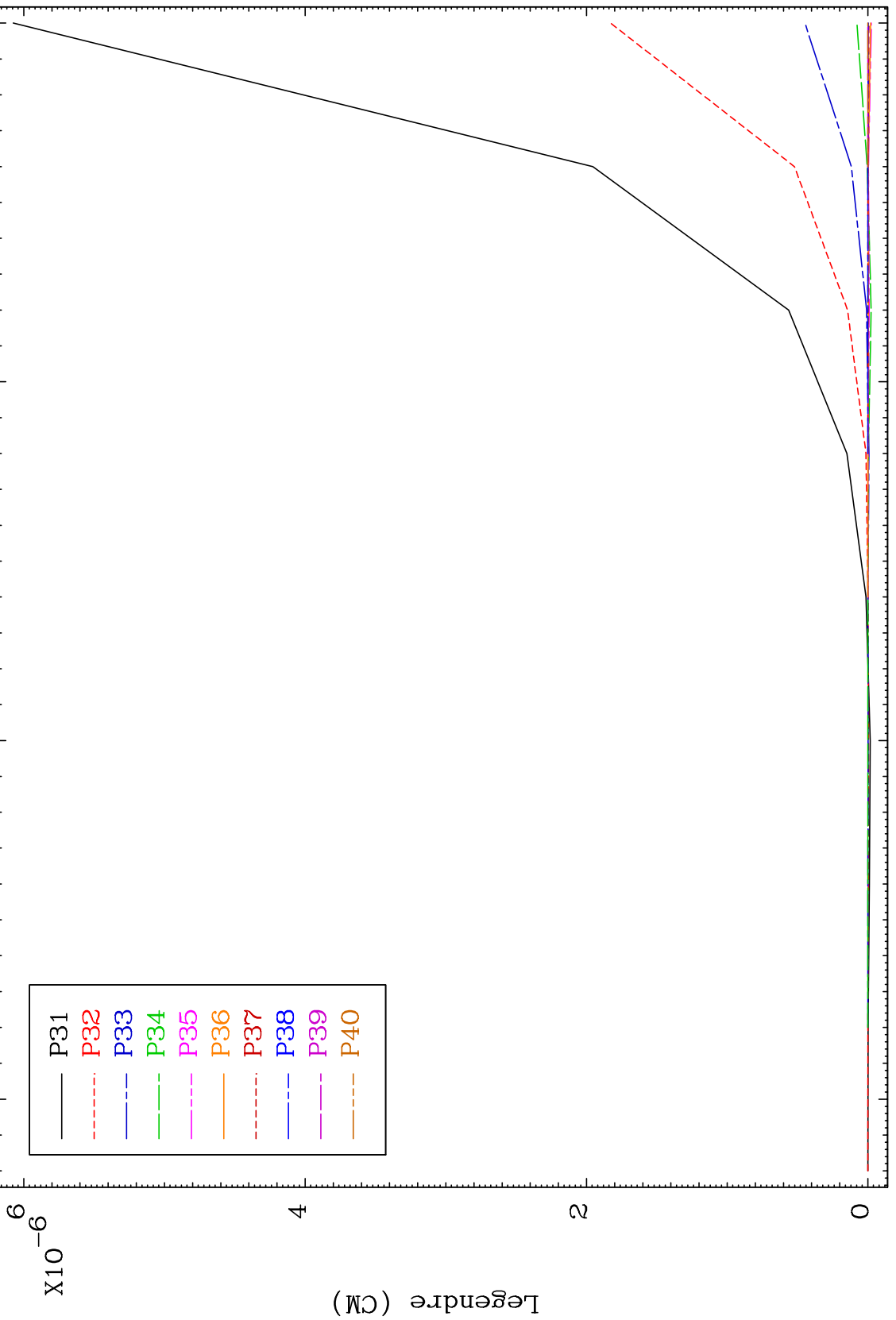
Incident Energy (MeV)

109-Mt-277

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Elastic Legendre Coefficients

109-Mt-277



19

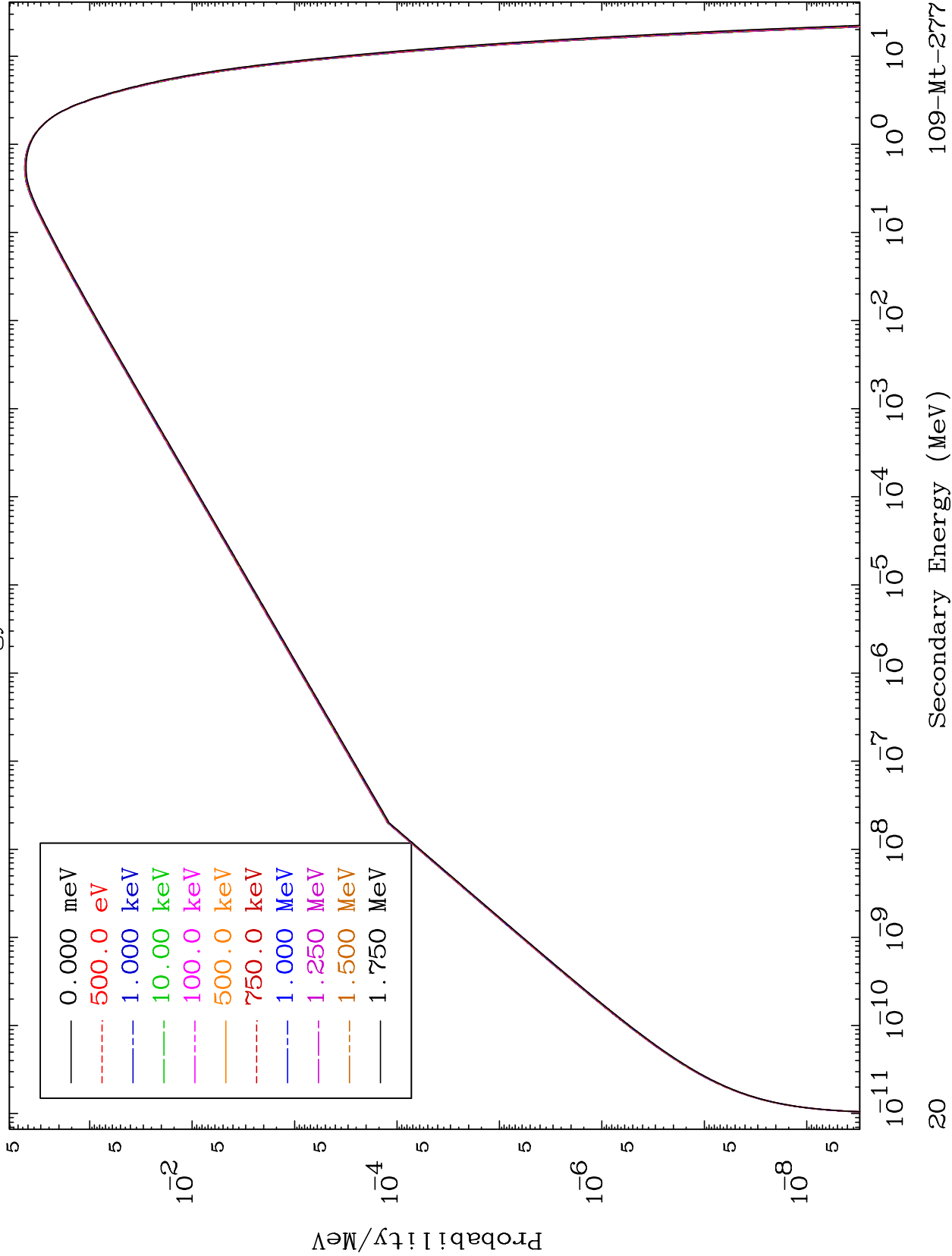
Incident Energy (MeV)

109-Mt-277

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### Fission Energy Distributions

109-Mt-277



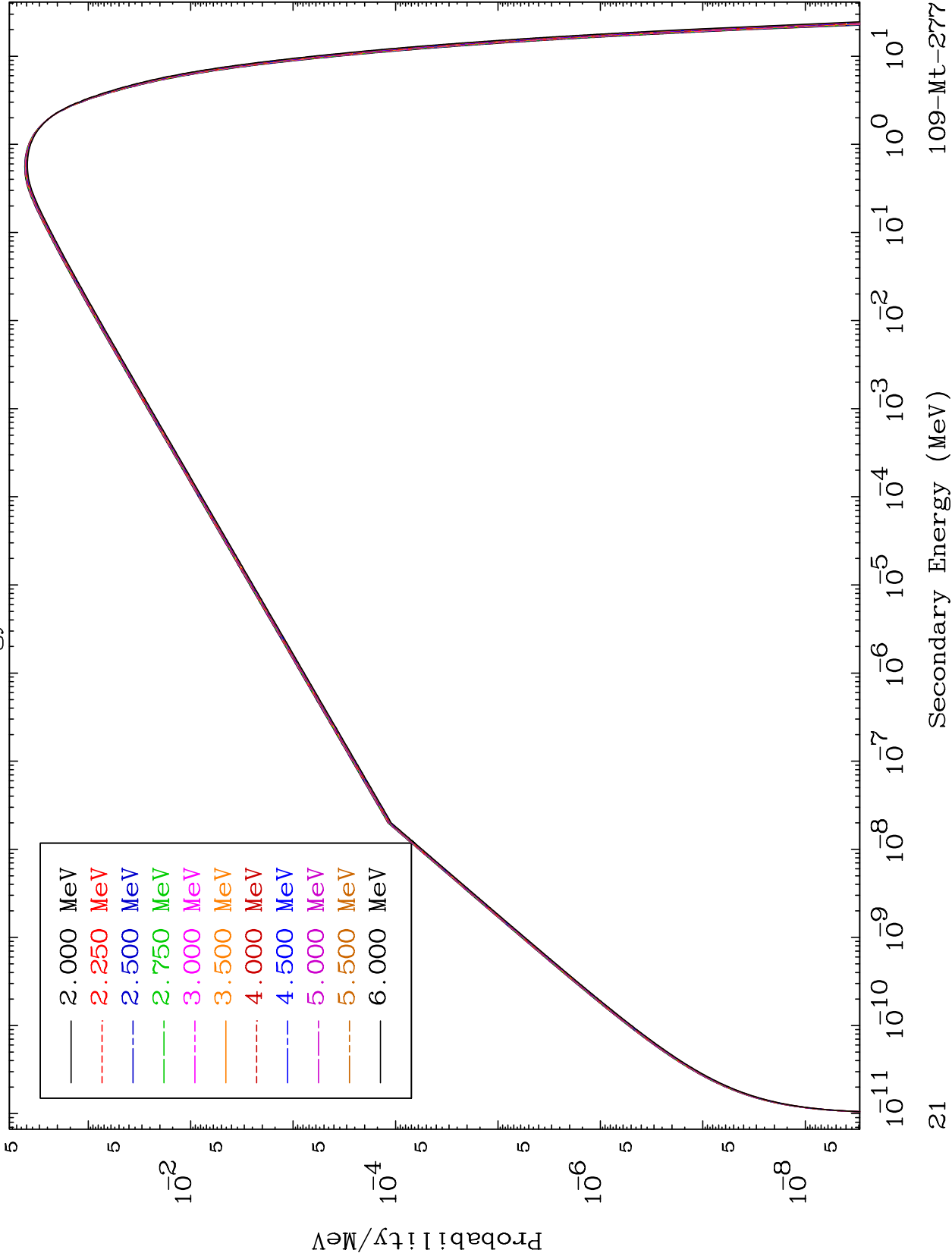
109-Mt-277

Secondary Energy (MeV)

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### Fission Energy Distributions

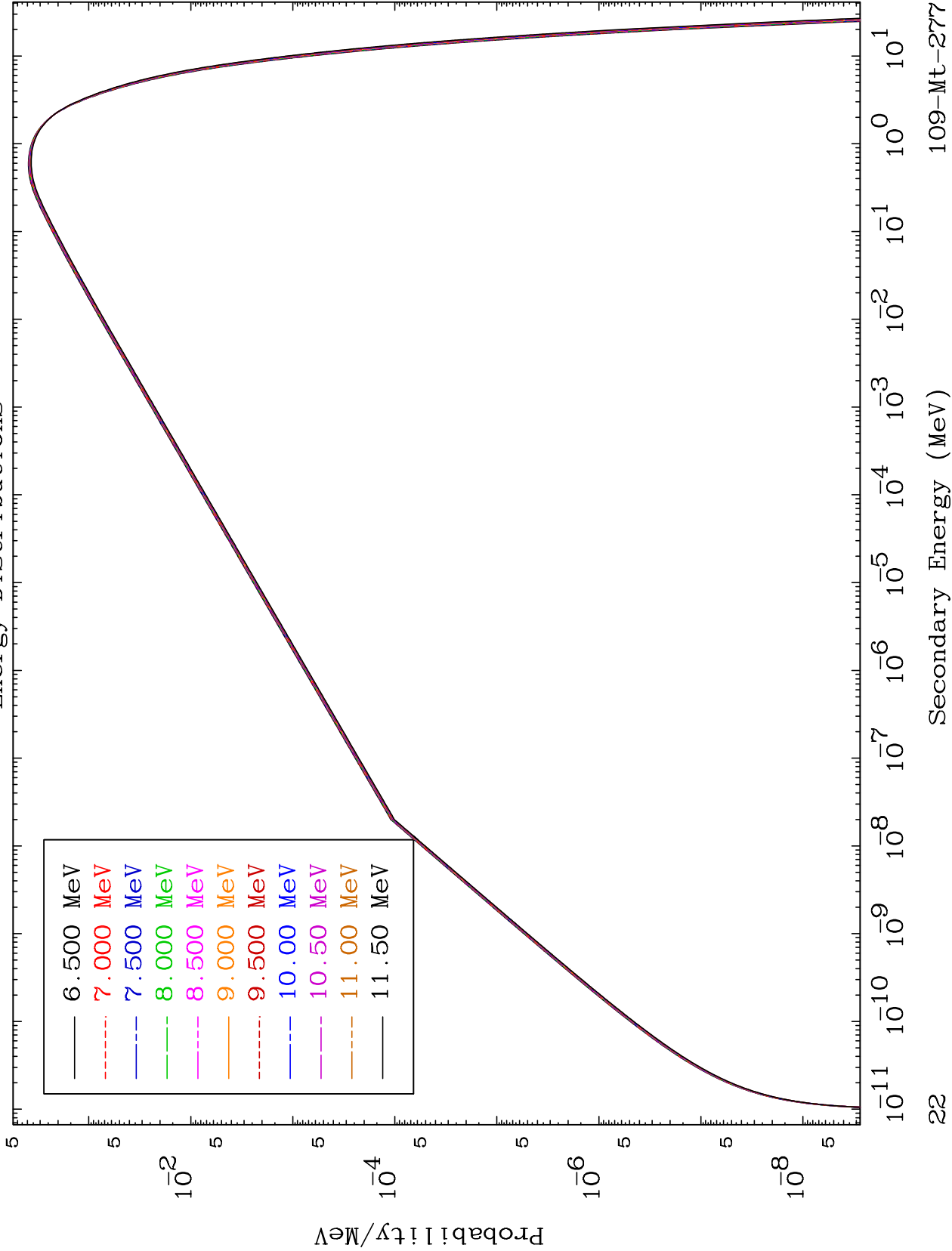
109-Mt-277



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### Fission Energy Distributions

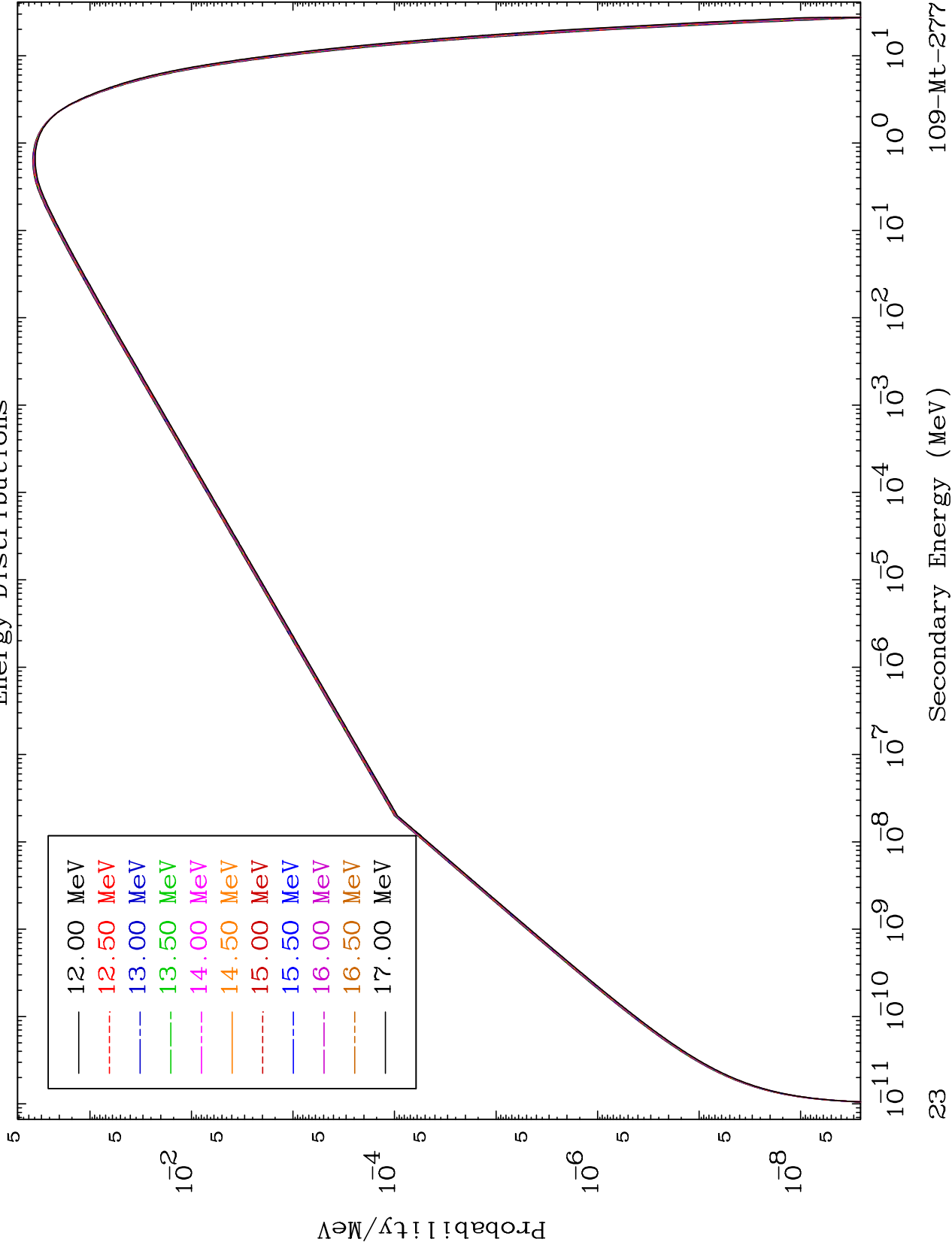
109-Mt-277



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### Fission Energy Distributions

109-Mt-277

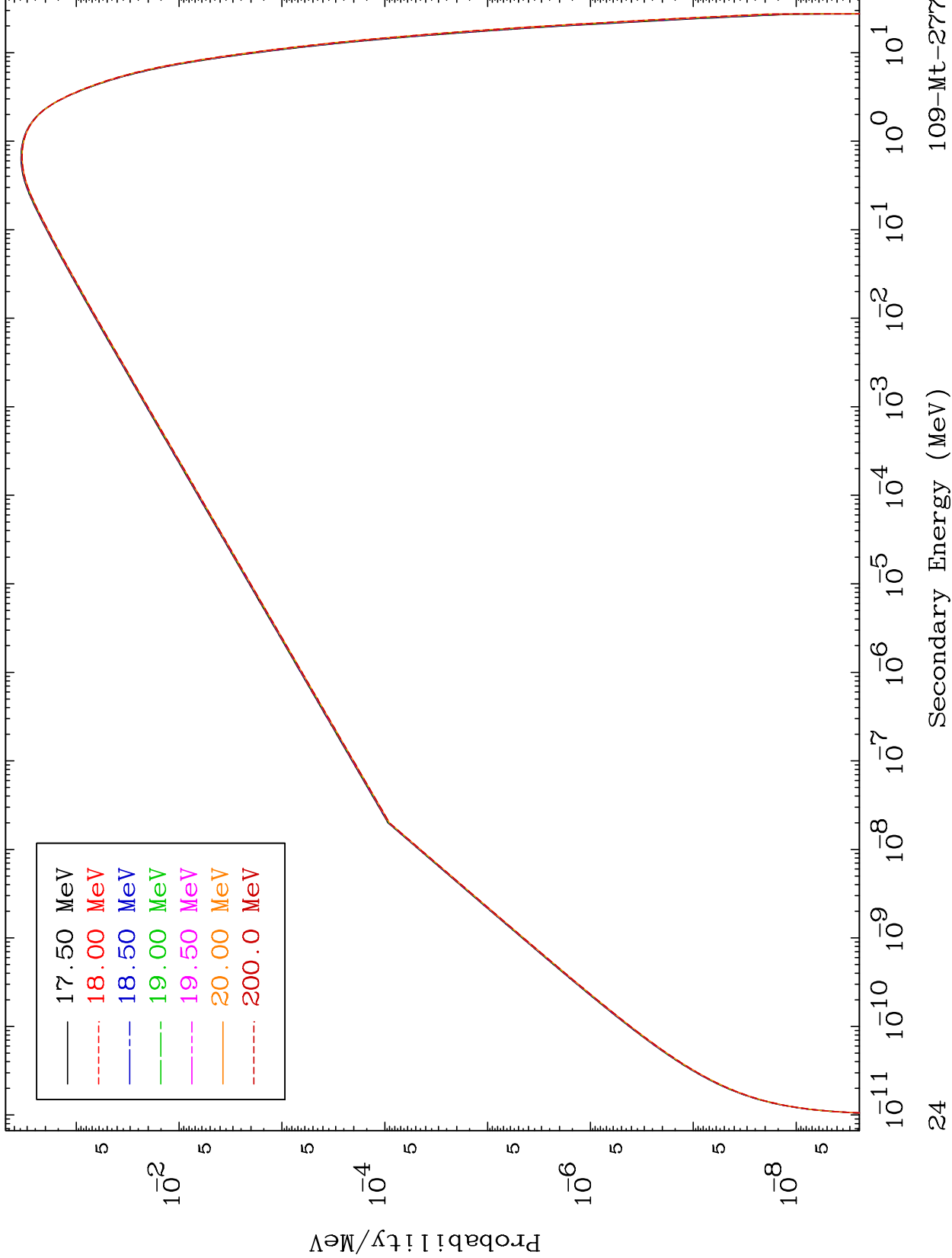




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### Fission Energy Distributions

109-Mt-277



109-Mt-277

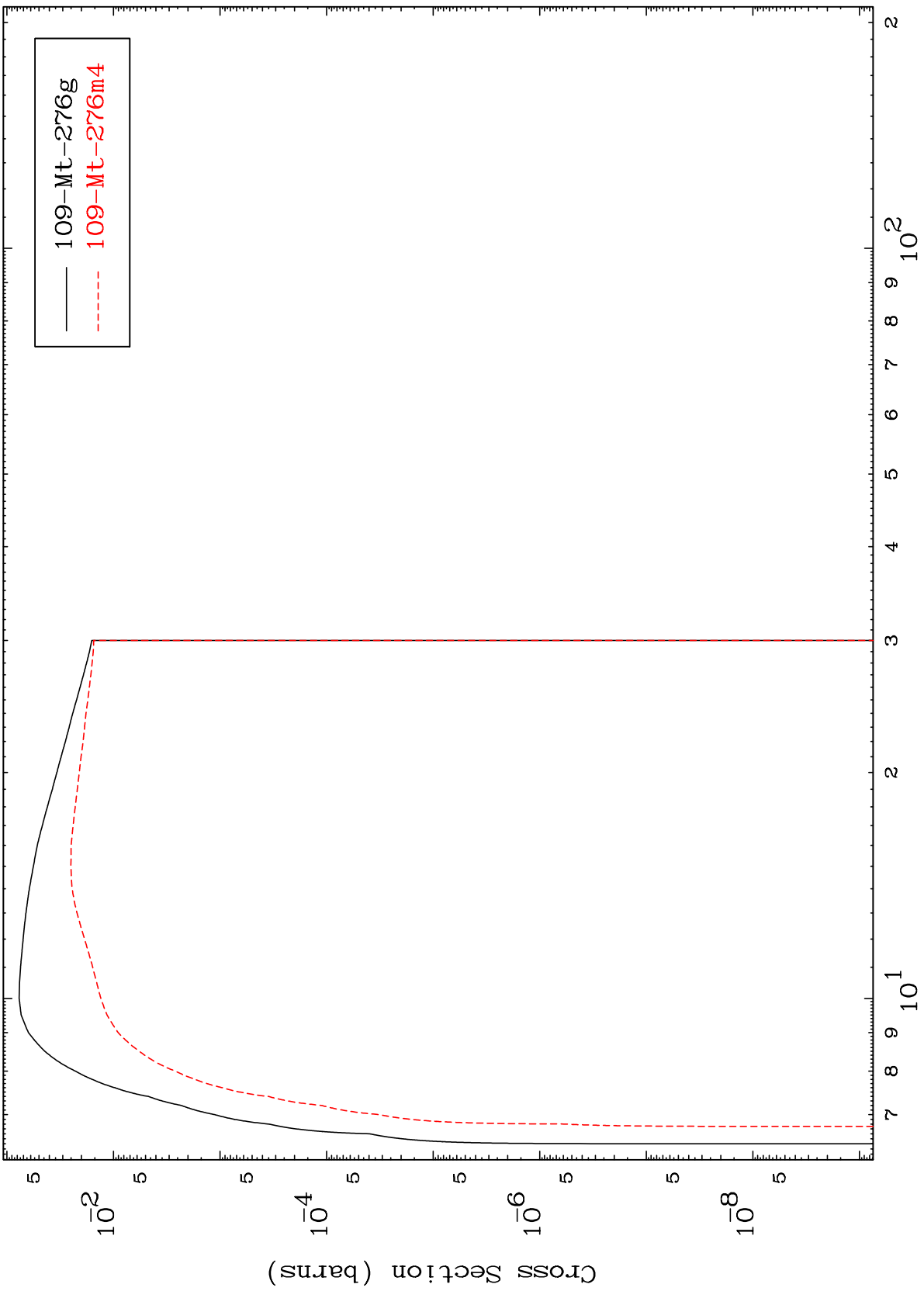
Secondary Energy (MeV)

24

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109-Mt-277

(n,2n)  
Radionuclide Production Cross Section



109-Mt-277

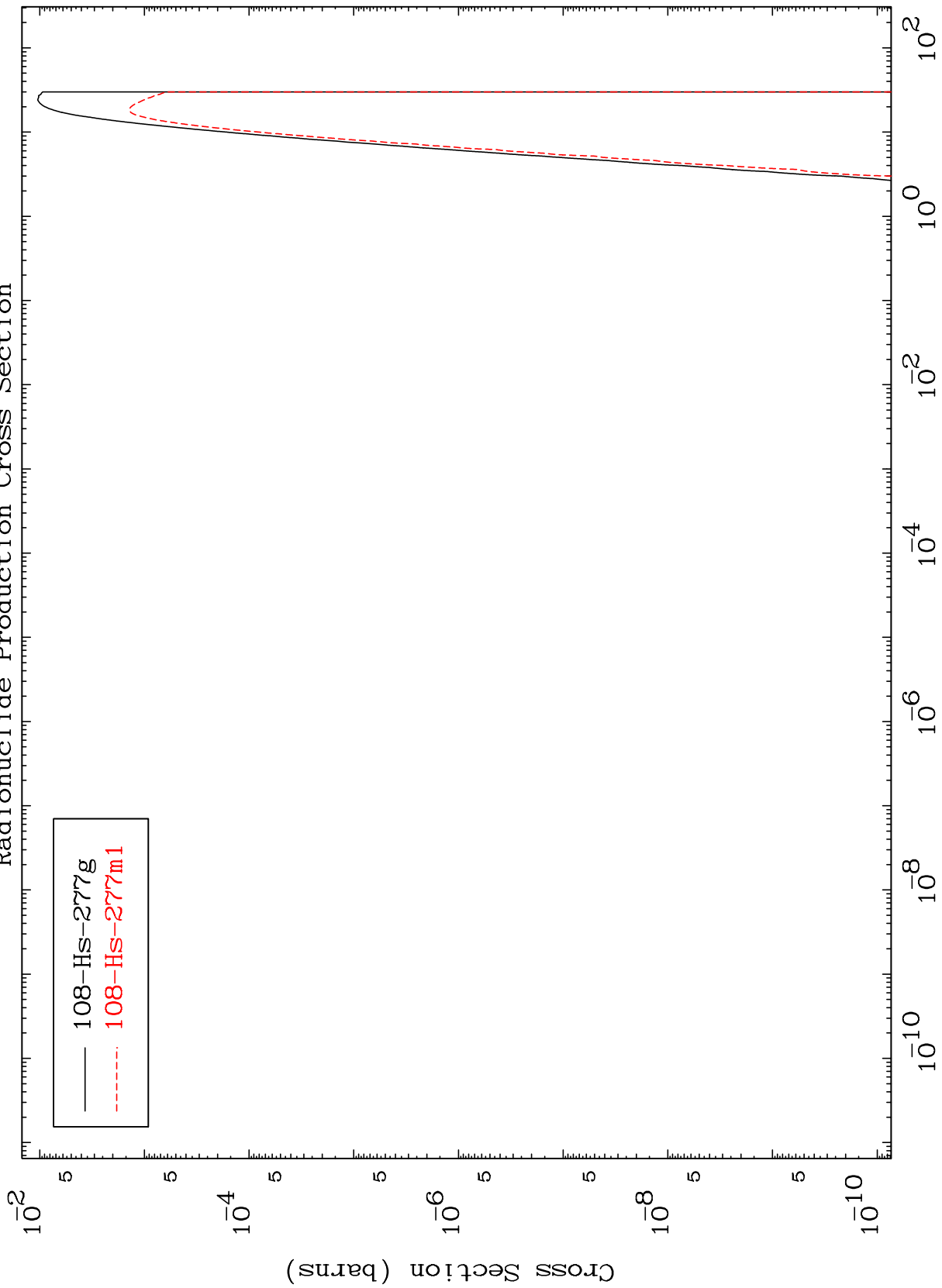
Incident Energy (MeV)

25

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109-Mt-277

Radionuclide Production Cross Section



— 108-Hs-277g  
- - - 108-Hs-277m1

109-Mt-277

Incident Energy (MeV)

26