

Program EVALPLOT  
(Version 2021-1)

by

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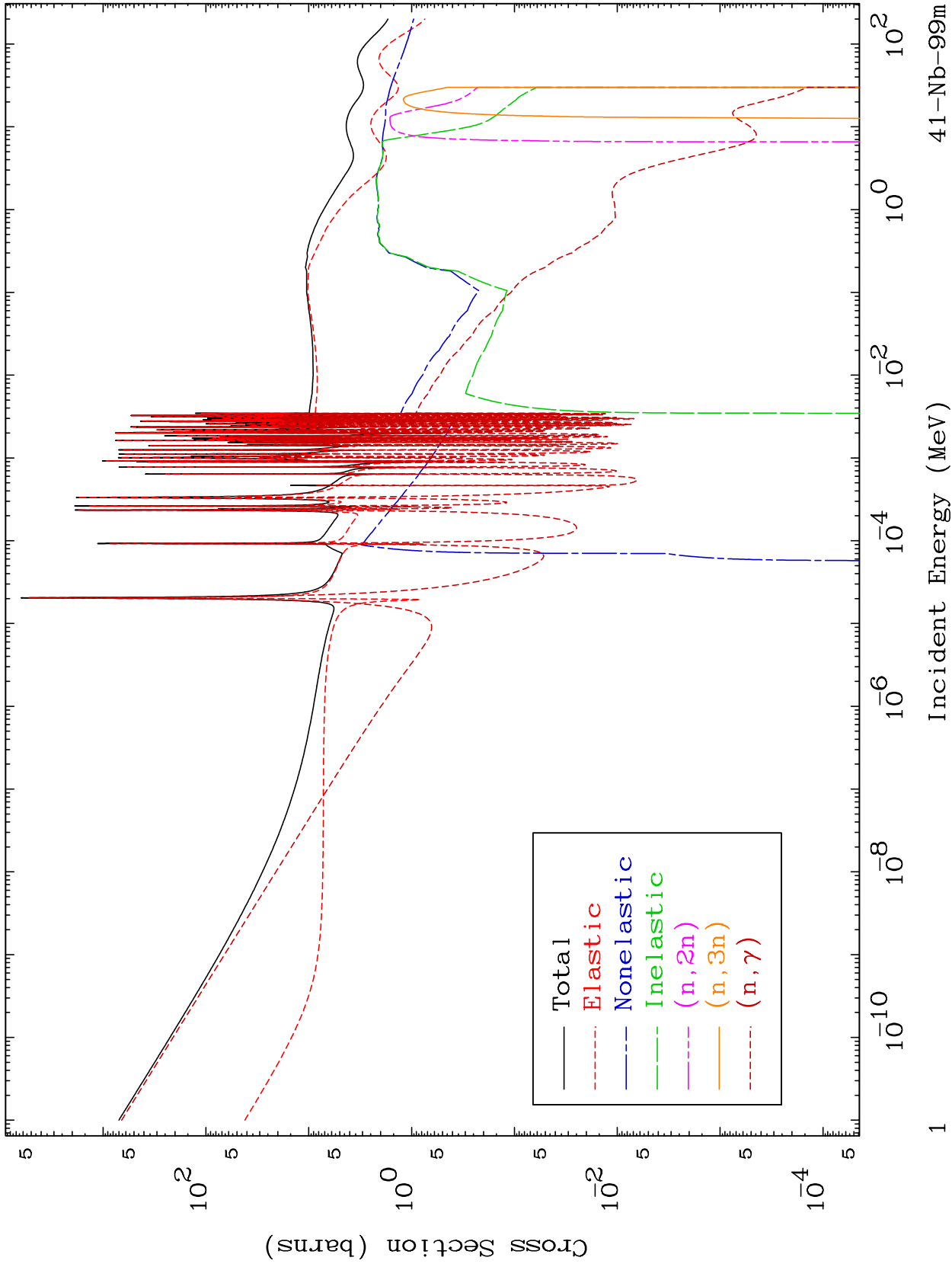
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Press Mouse Button to Start

MAT 4144

Neutron Major  
293 Kelvin Cross Sections

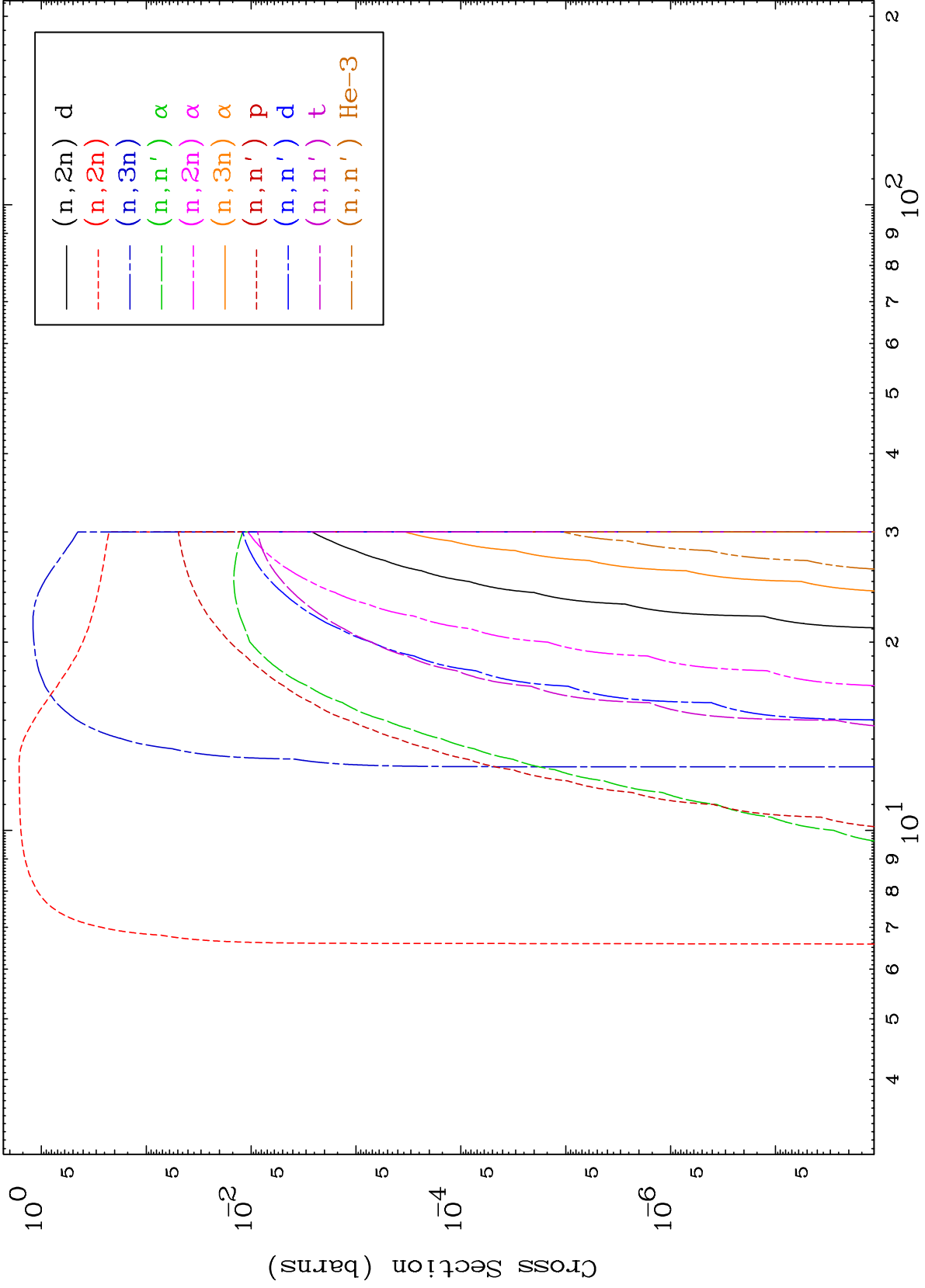
41-Nb-99m



MAT 4144

Neutron Absorption  
293 Kelvin Cross Sections

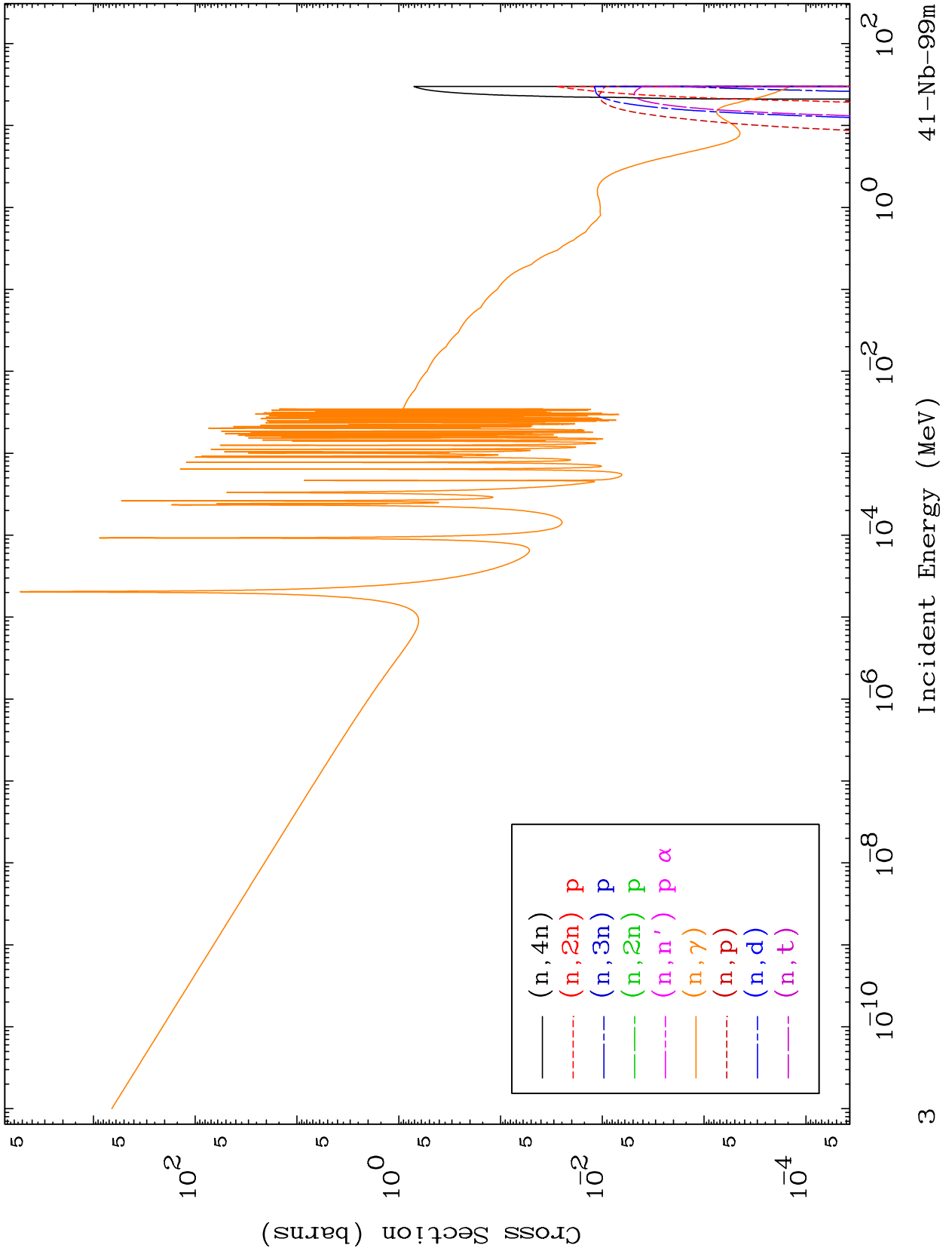
41-Nb-99m



MAT 4144

Neutron Absorption  
293 Kelvin Cross Sections

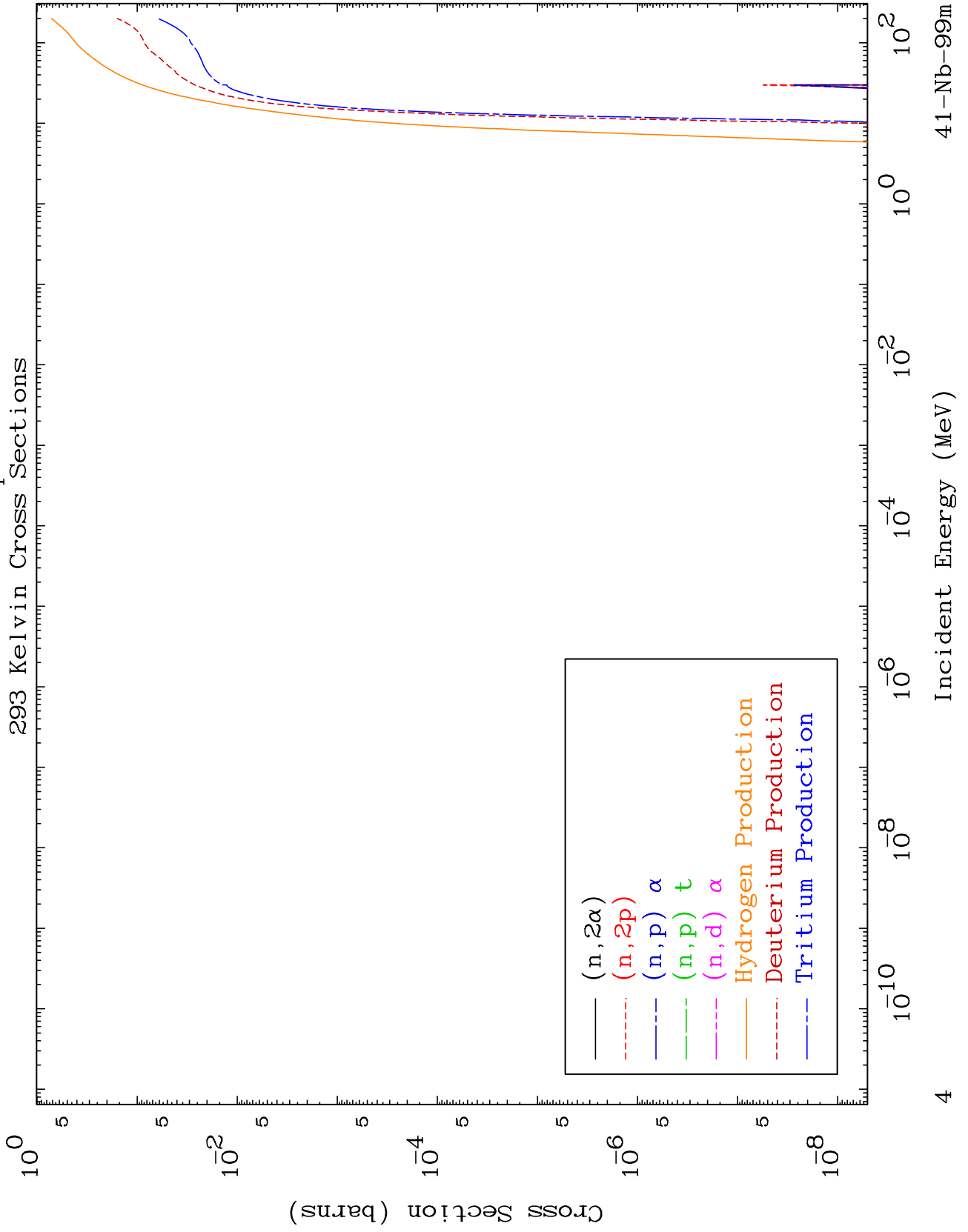
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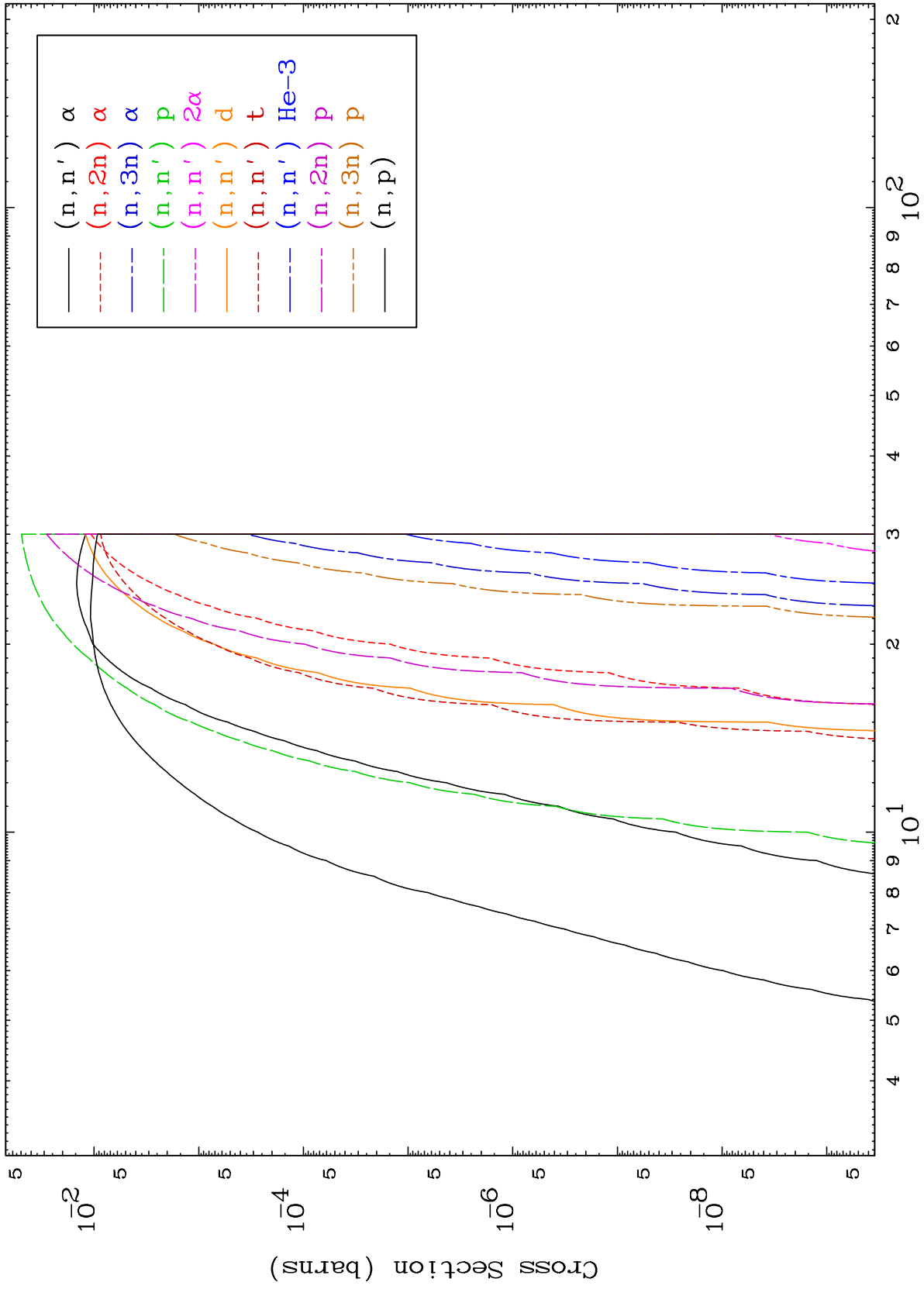


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Neutron Absorption  
293 Kelvin Cross Sections

41-Nb-99m

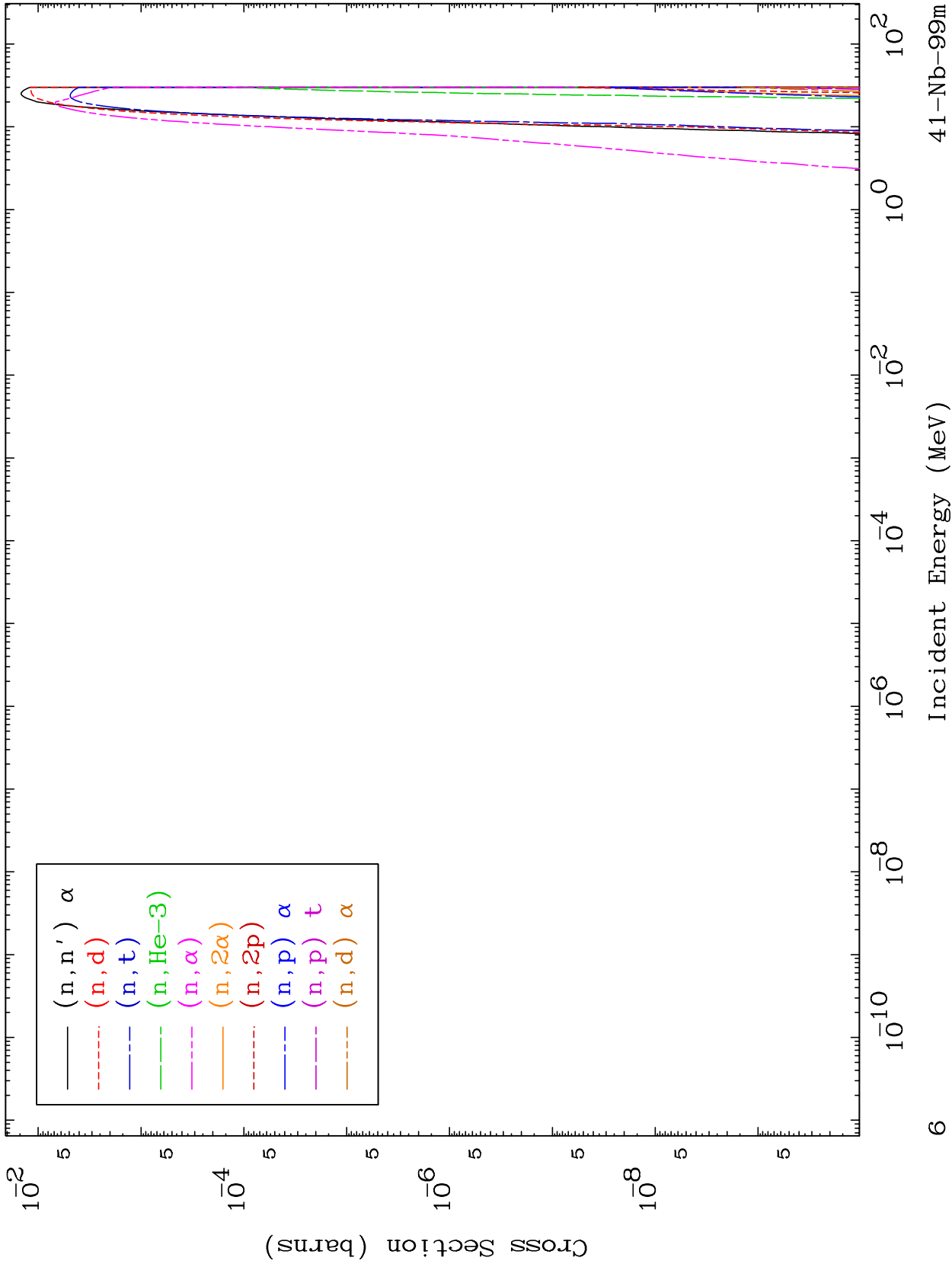




MAT 4144

Charged Particle  
293 Kelvin Cross Sections

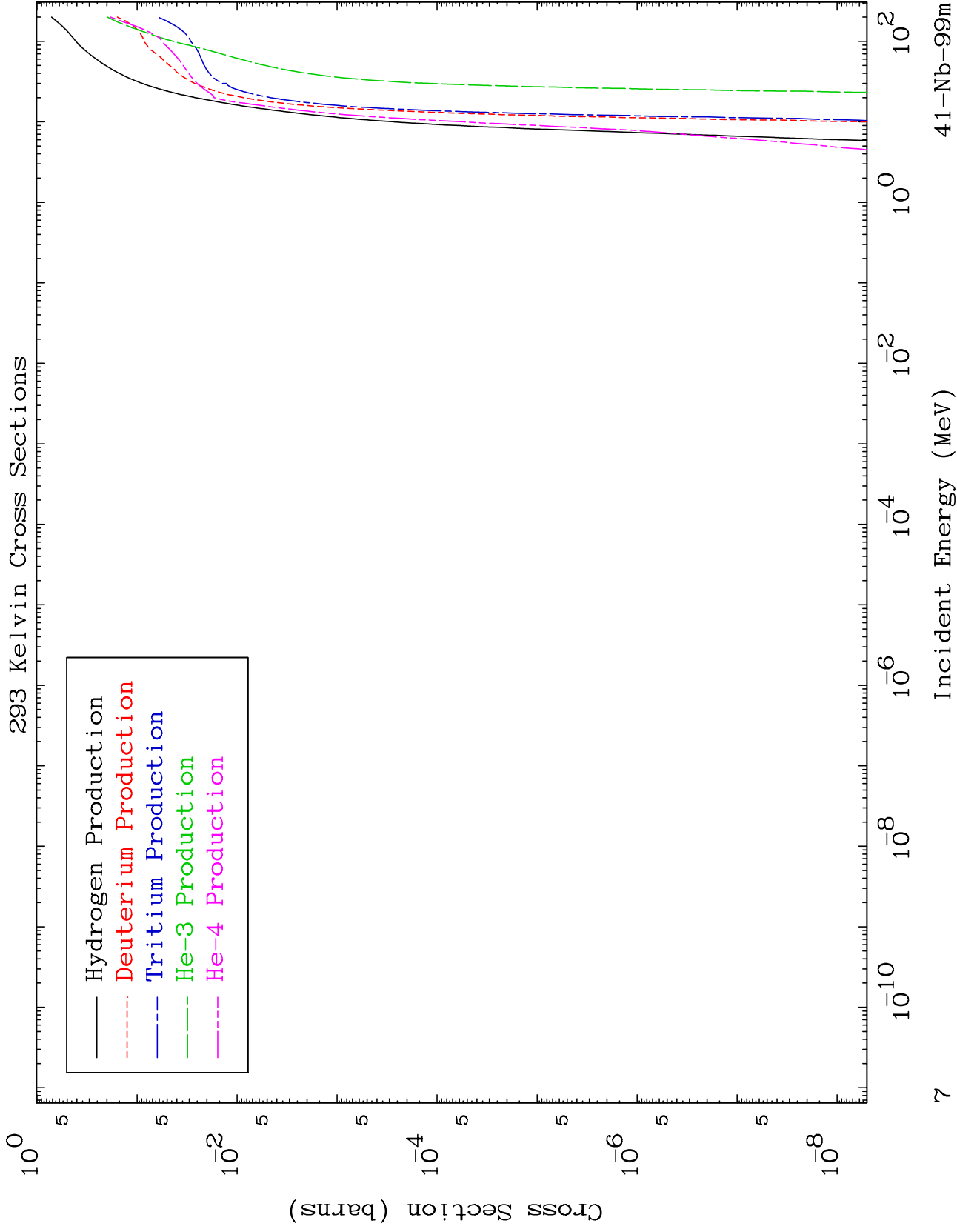
41-Nb-99m



MAT 4144

Particle Production  
293 Kelvin Cross Sections

41-Nb-99m

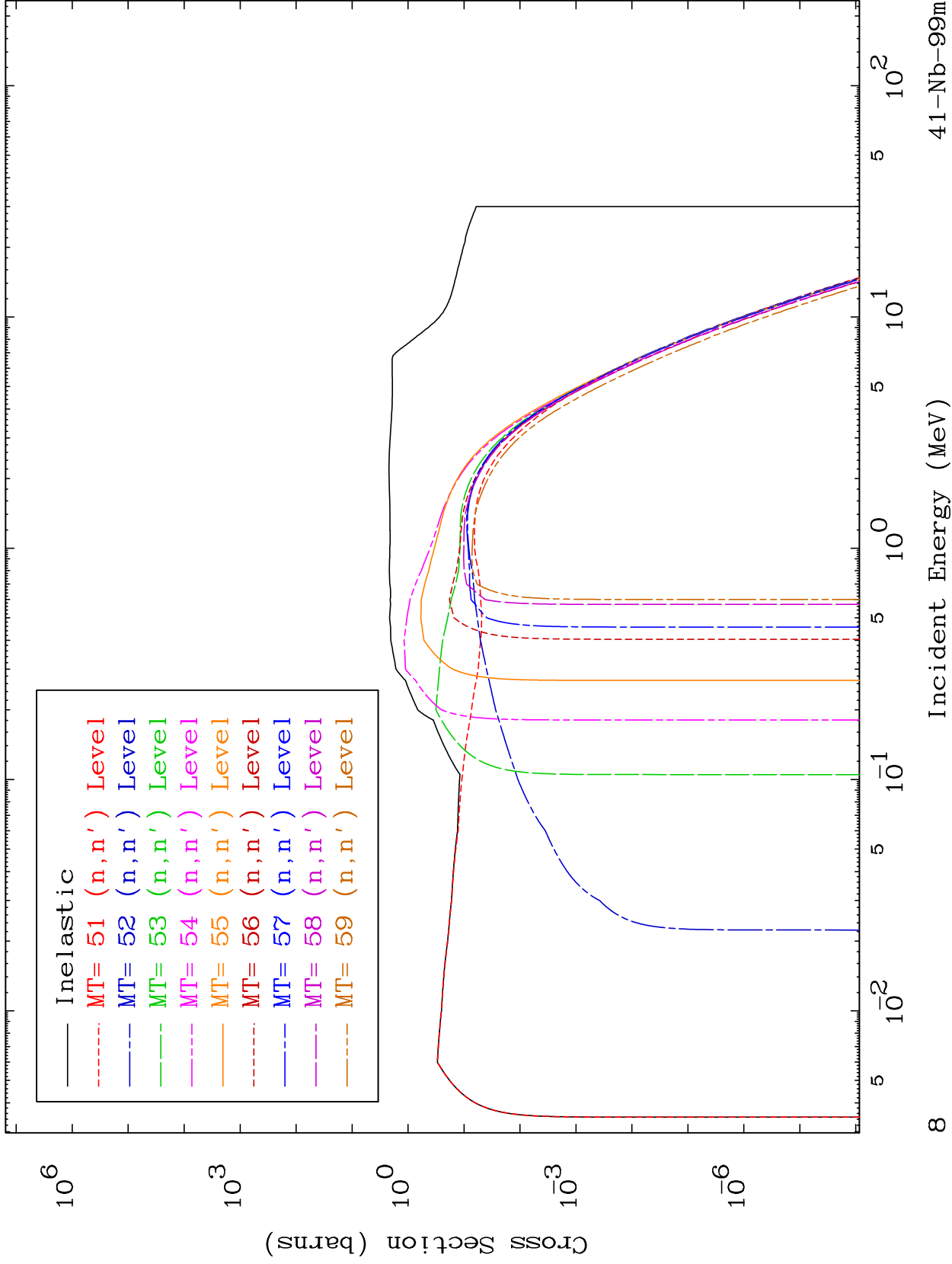




MAT 4144

(n,n') Levels  
293 Kelvin Cross Sections

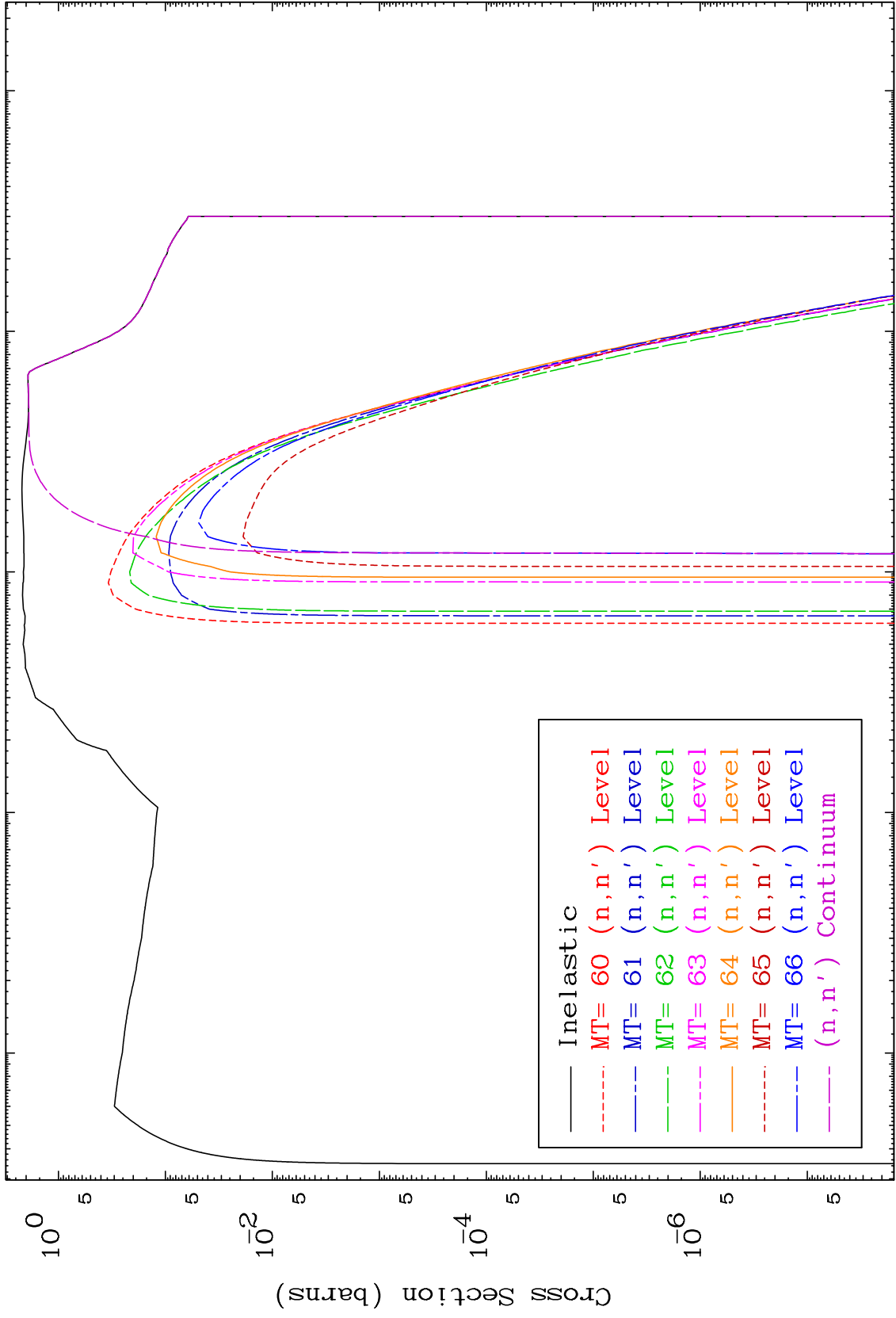
41-Nb-99m



MAT 4144

293 Kelvin Cross Sections  
(n,n') Levels

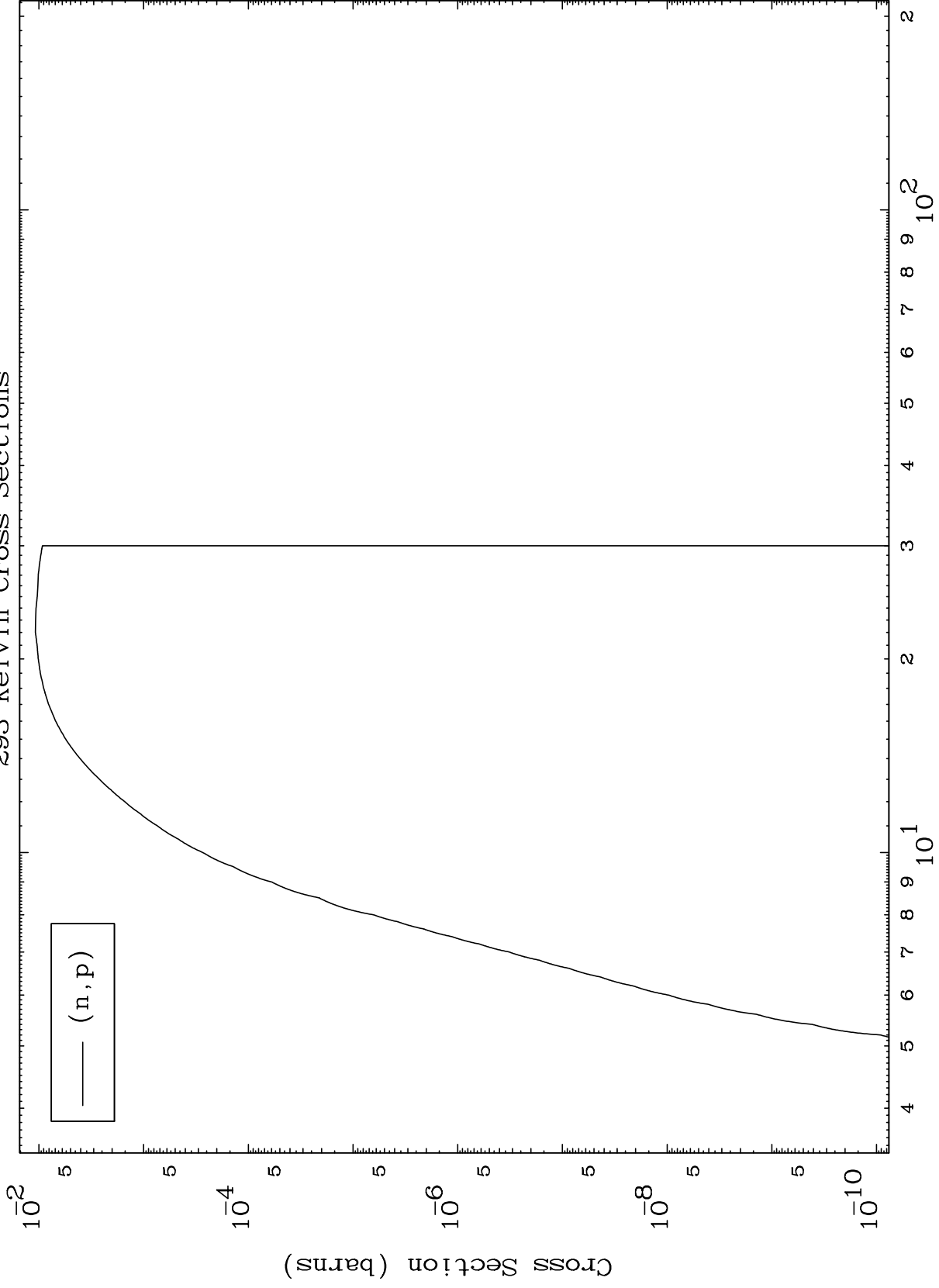
41-Nb-99m



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(n,p) Levels  
293 Kelvin Cross Sections

41-Nb-99m



10

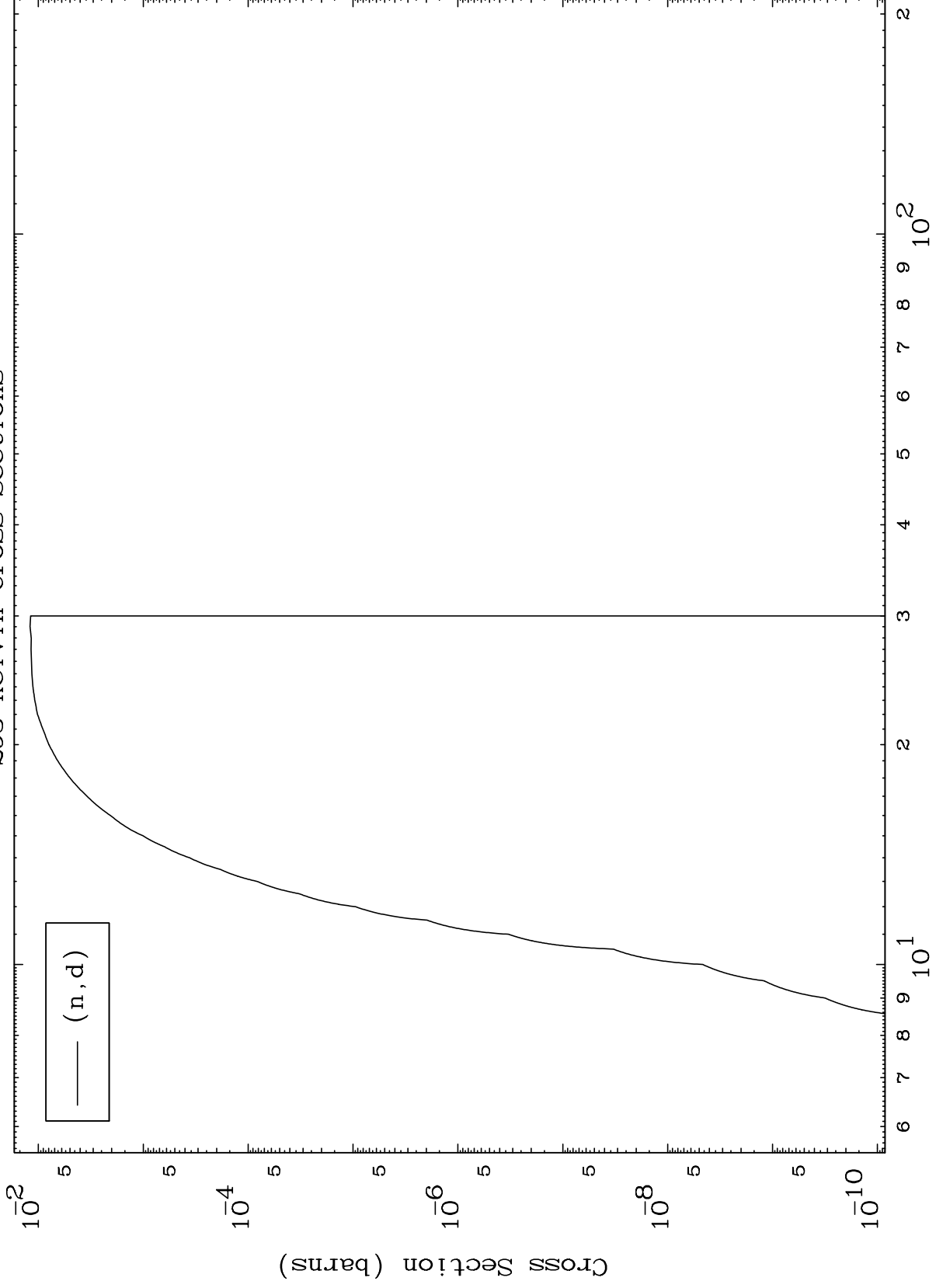
Incident Energy (MeV)

41-Nb-99m

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(n,d) Levels  
293 Kelvin Cross Sections

41-Nb-99m



11

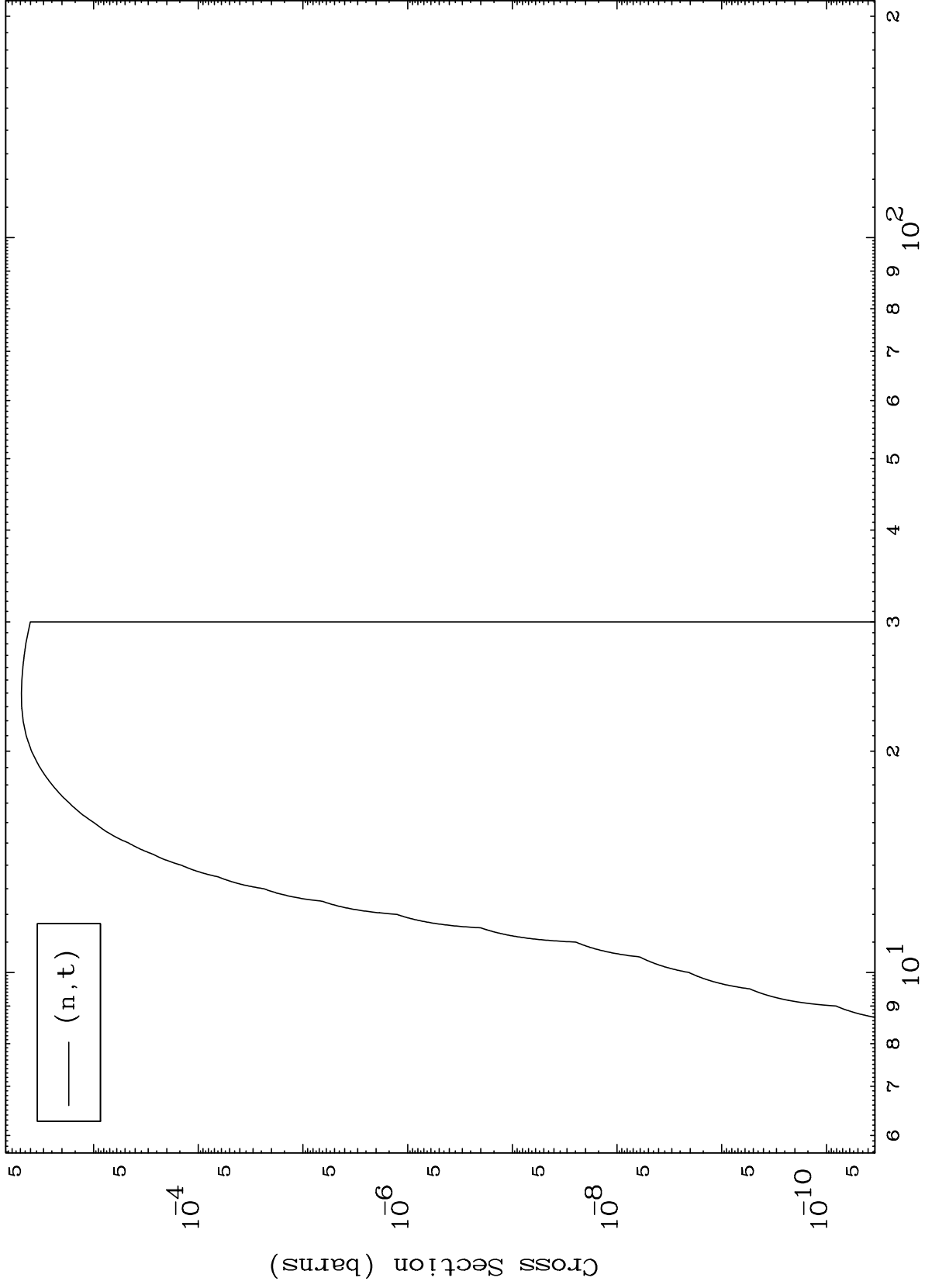
Incident Energy (MeV)

41-Nb-99m

MAT 4144

(n, t) Levels  
293 Kelvin Cross Sections

41-Nb-99m



12

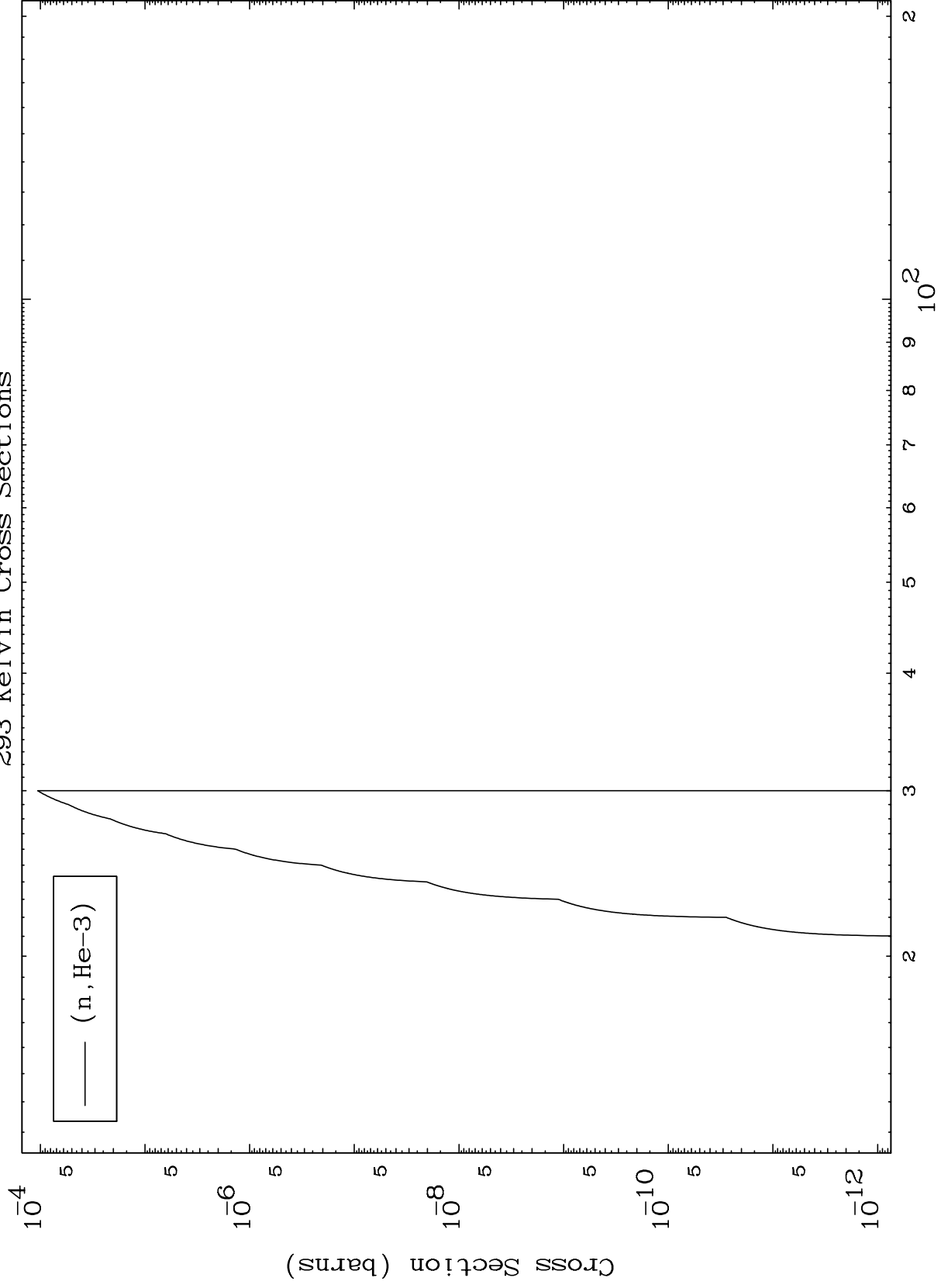
Incident Energy (MeV)

41-Nb-99m

MAT 4144

(n,He3) Levels  
293 Kelvin Cross Sections

41-Nb-99m



13

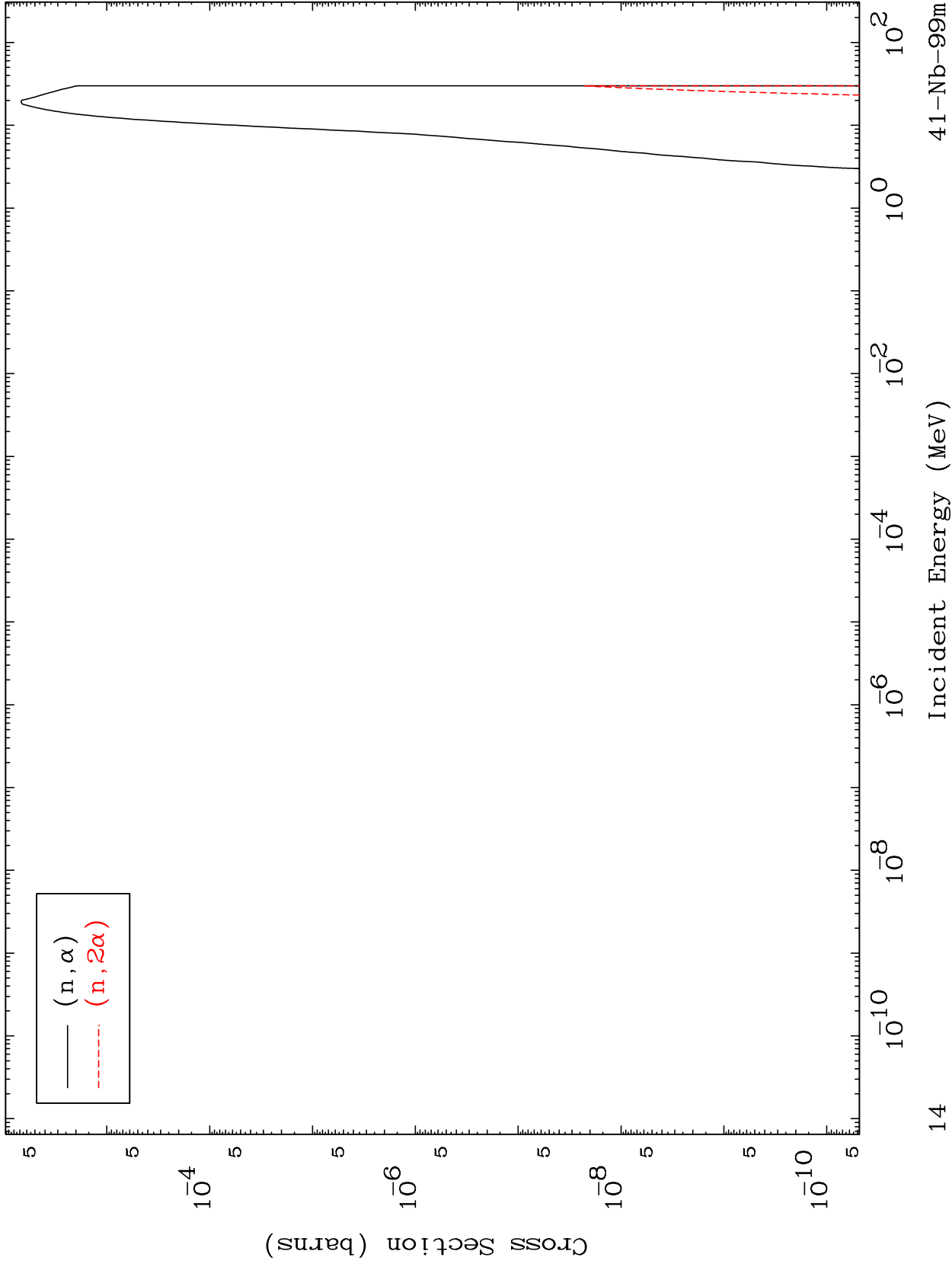
Incident Energy (MeV)

41-Nb-99m

MAT 4144

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

41-Nb-99m



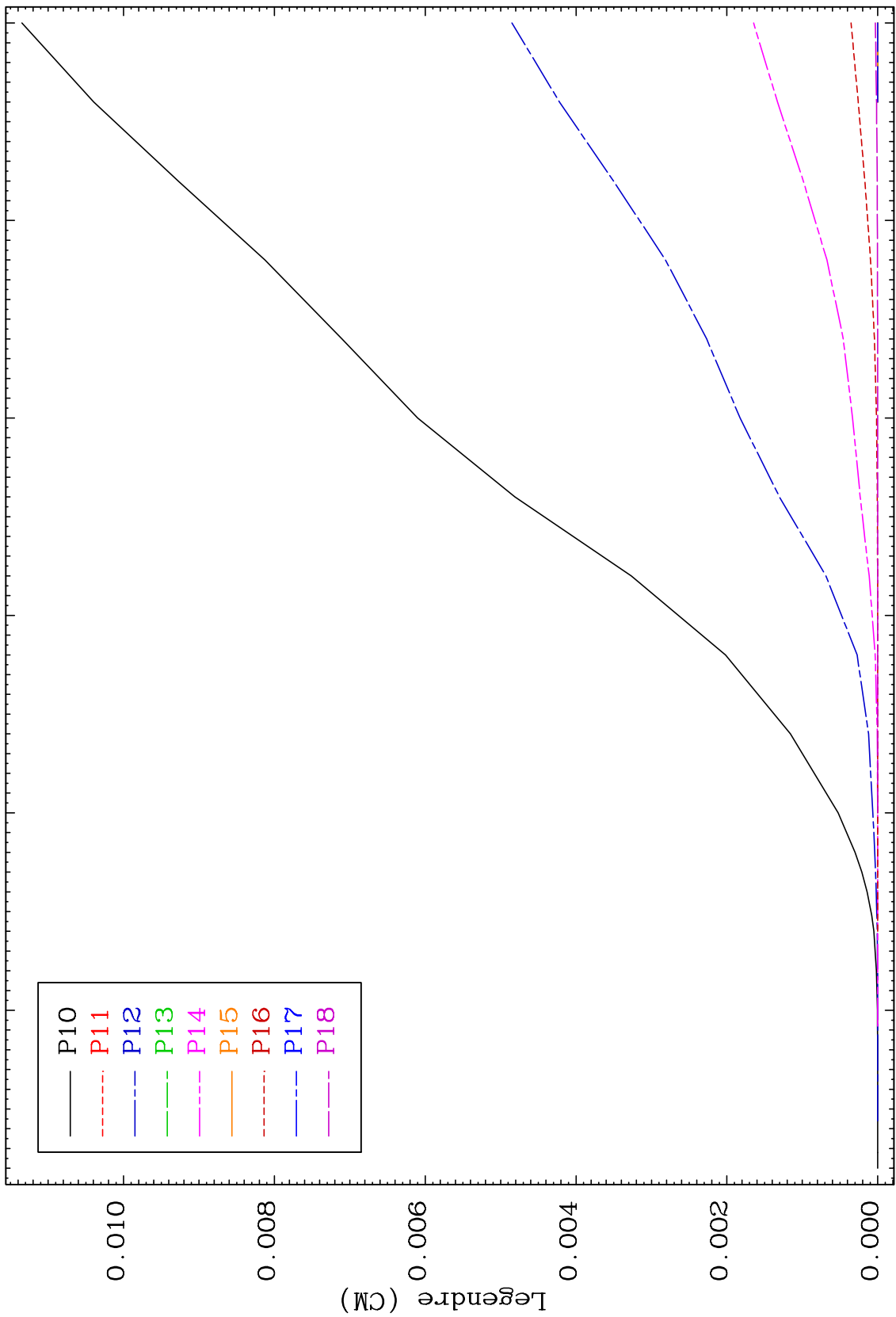




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### Elastic Legendre Coefficients

41-Nb-99m



16

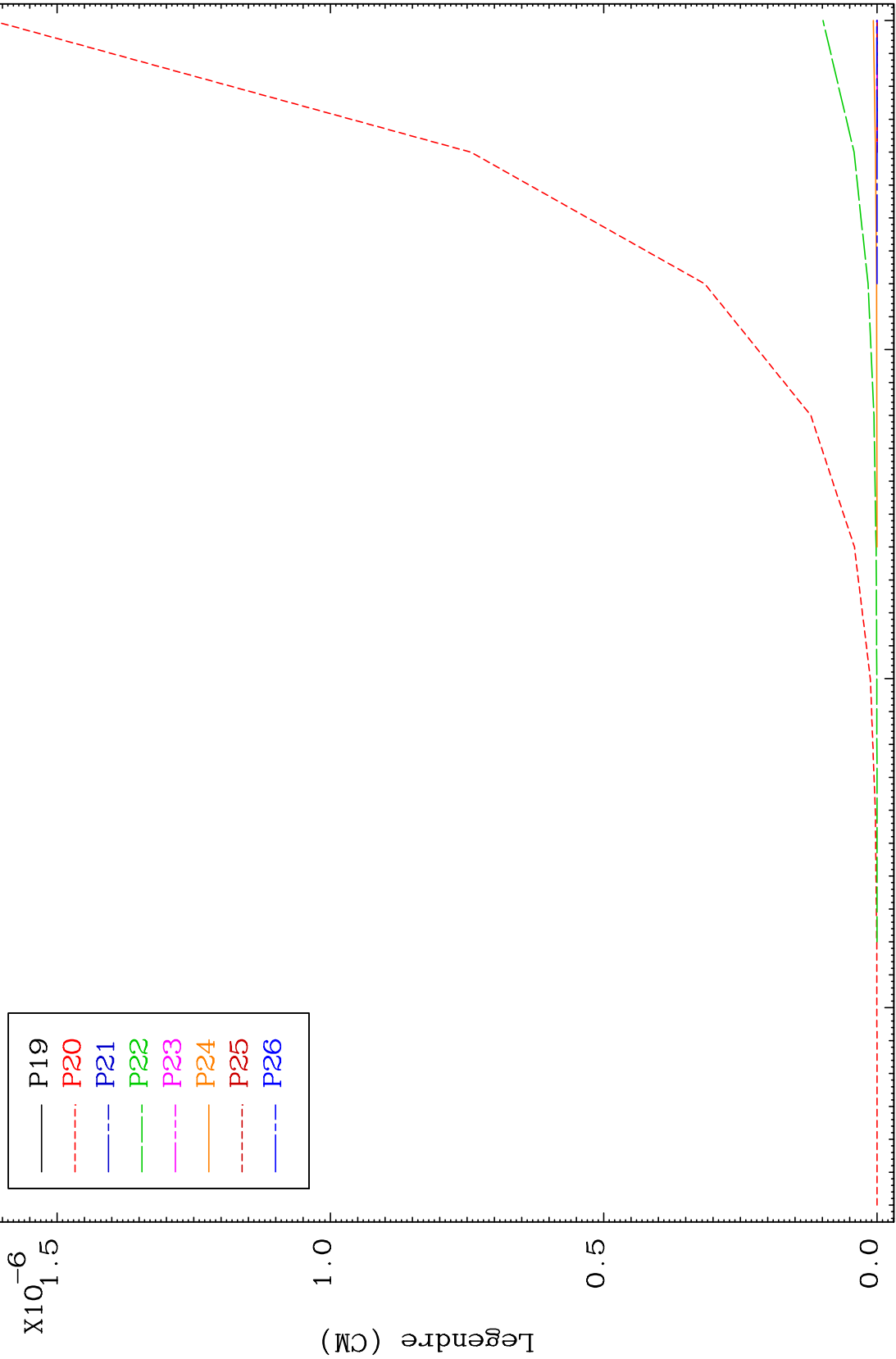
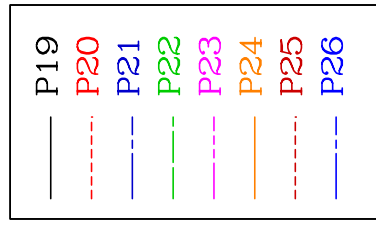
41-Nb-99m

Incident Energy (MeV)

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### Elastic Legendre Coefficients

41-Nb-99m



17

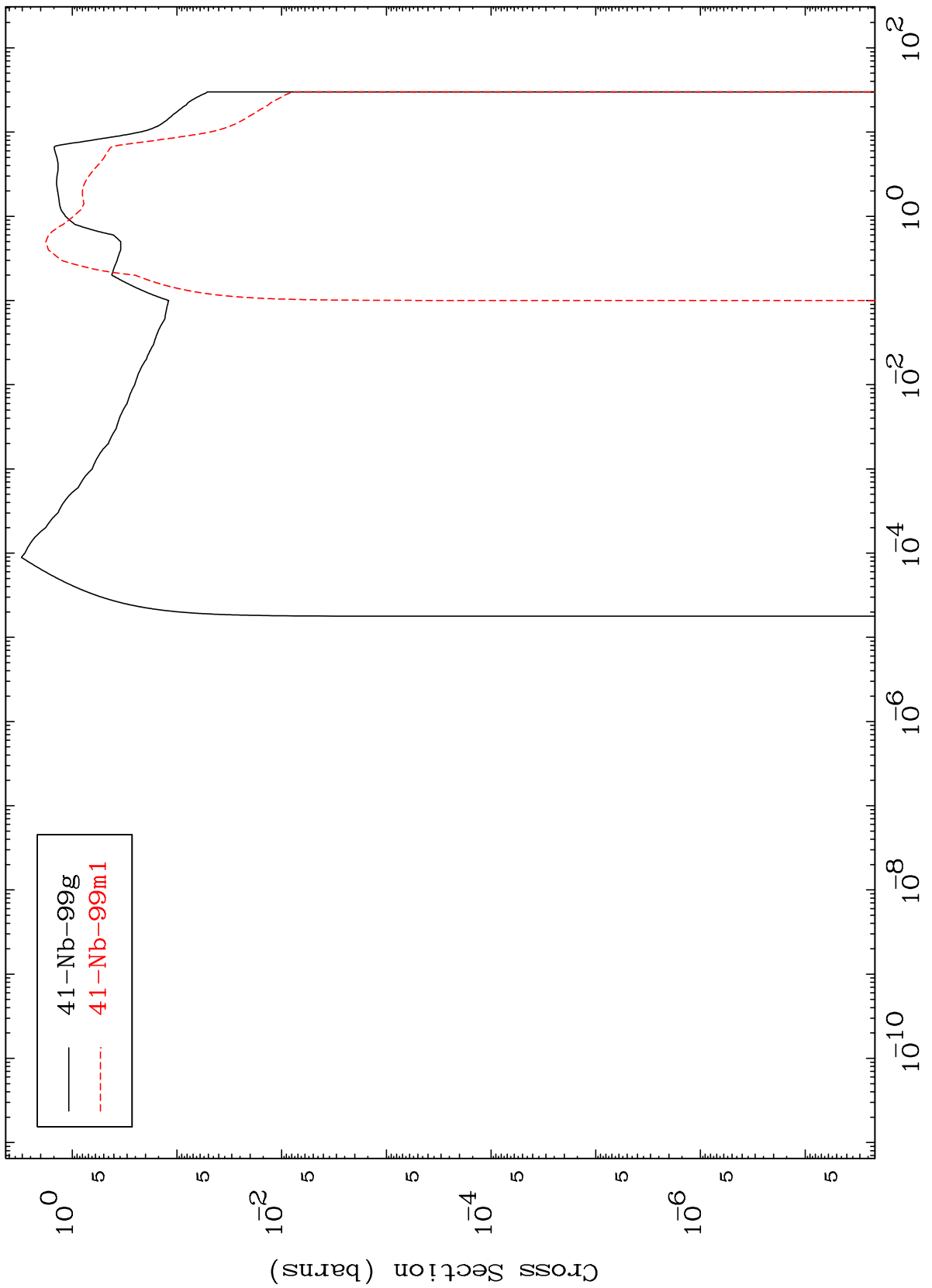
Incident Energy (MeV)

41-Nb-99m

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41-Nb-99m

Inelastic  
Radionuclide Production Cross Section

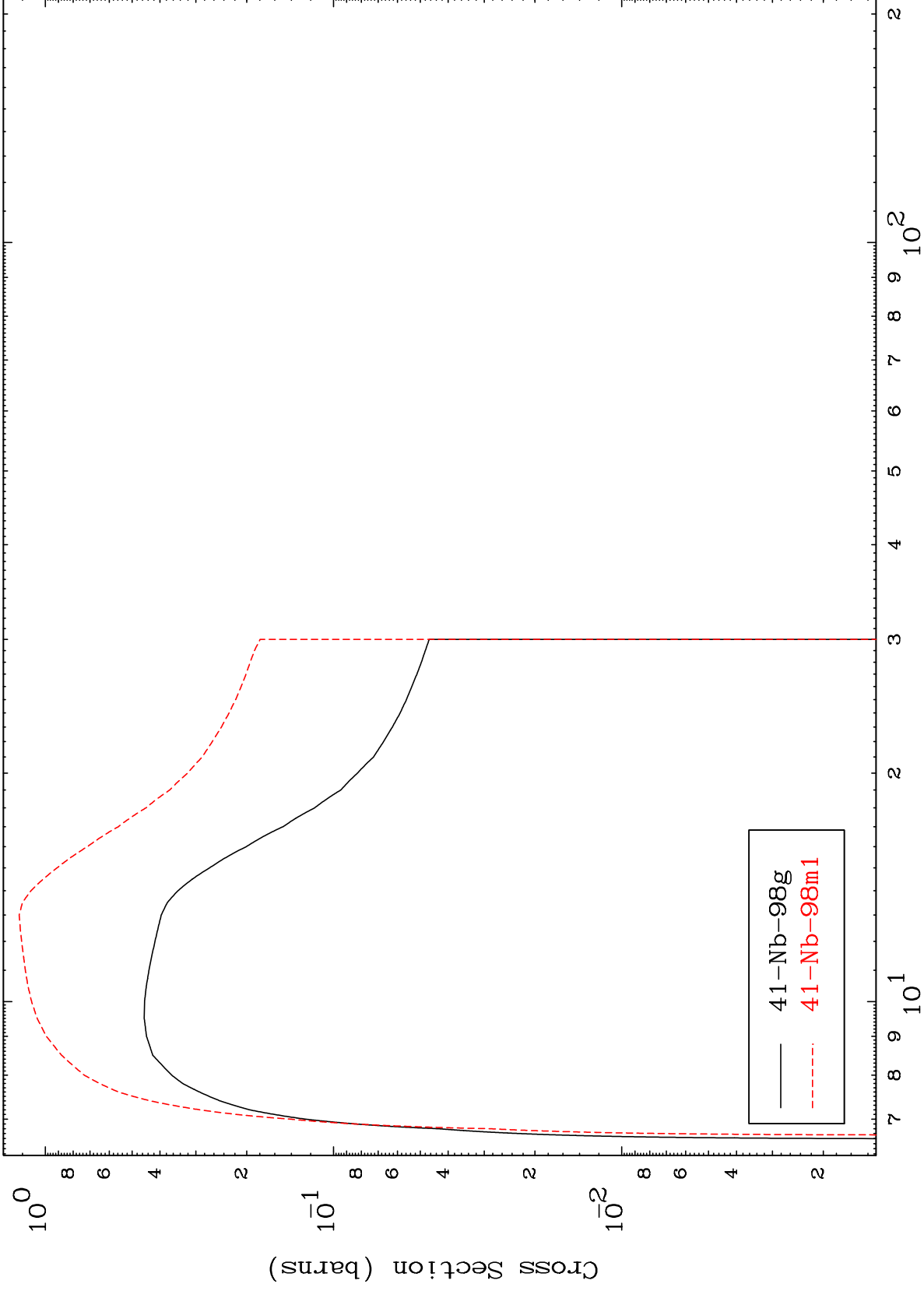


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(n,2n)

41-Nb-99m

Radionuclide Production Cross Section



19

Incident Energy (MeV)

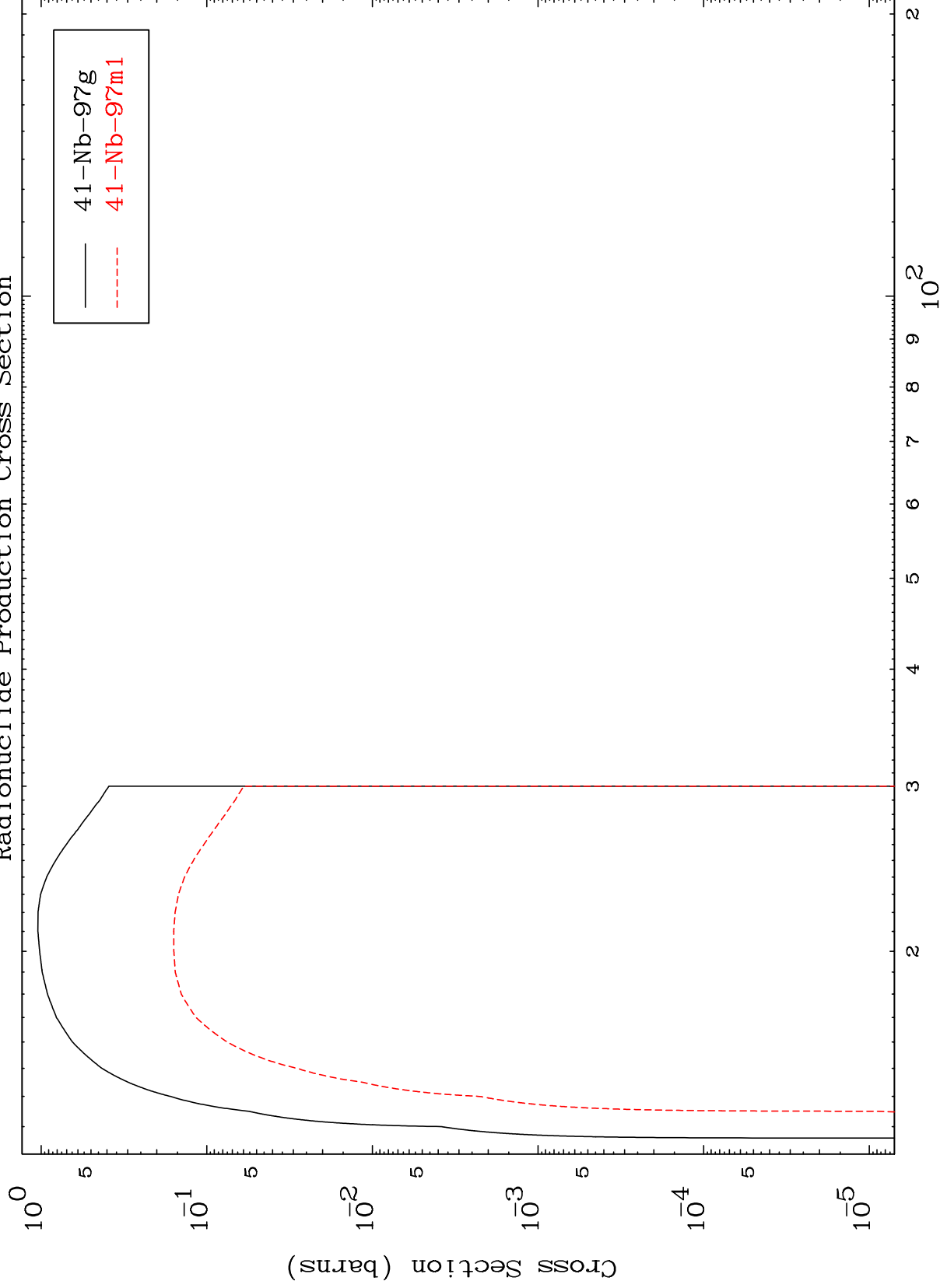
41-Nb-99m

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(n,3n)

41-Nb-99m

Radionuclide Production Cross Section



20

Incident Energy (MeV)

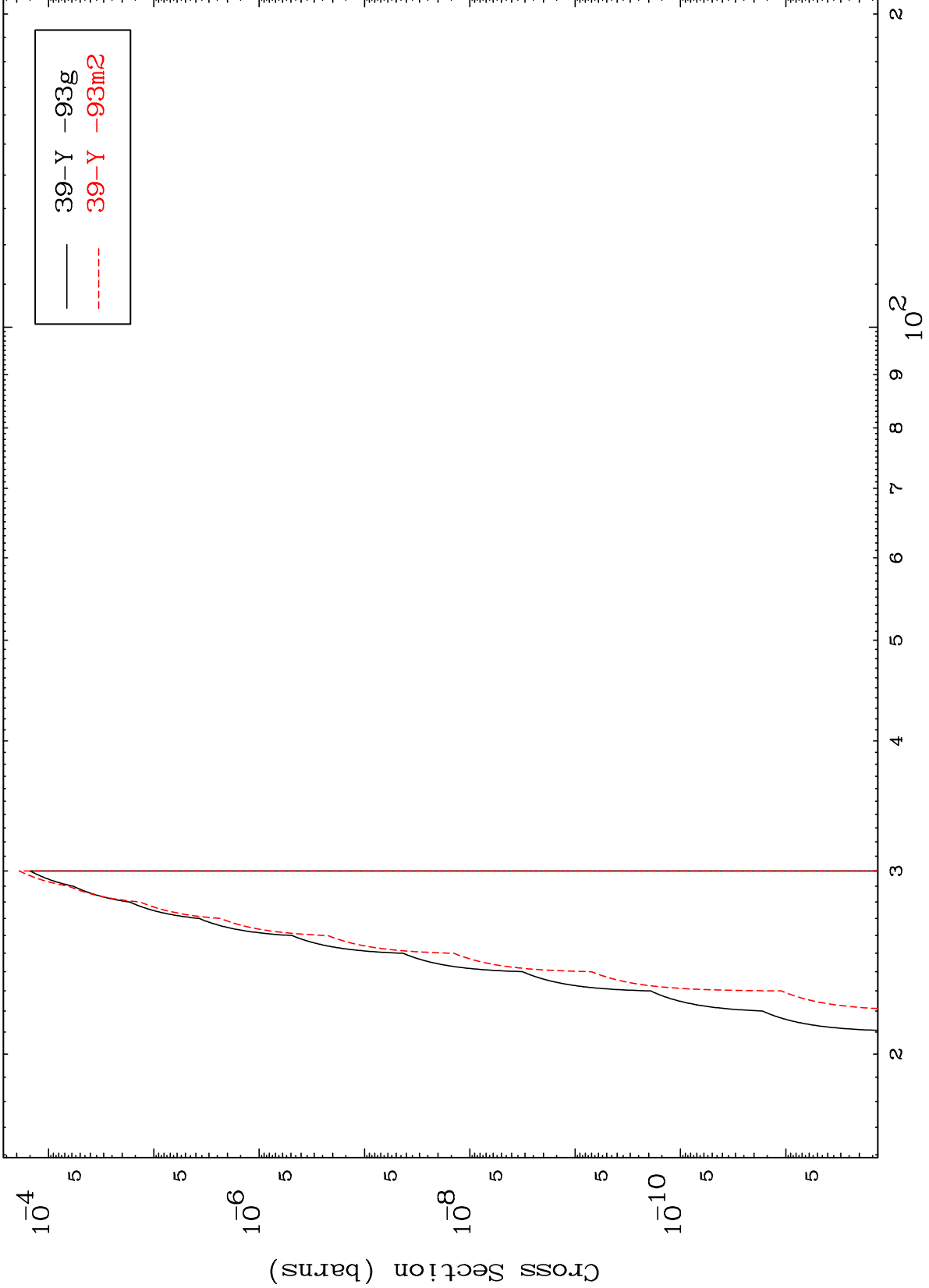
41-Nb-99m

MAT 4144

(n,3n)  $\alpha$

41-Nb-99m

Radionuclide Production Cross Section

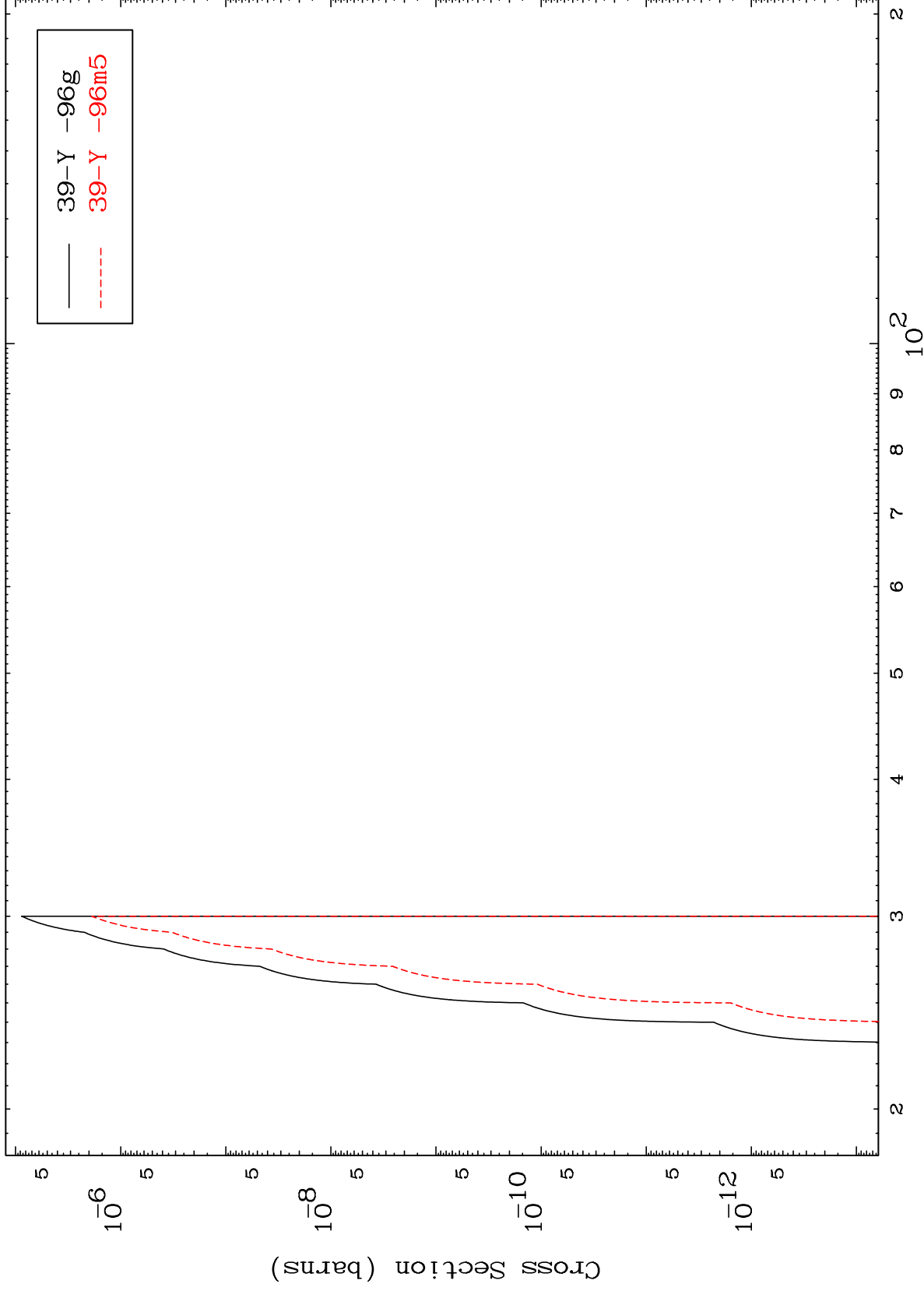


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(n,n') He-3

41-Nb-99m

Radionuclide Production Cross Section



22

Incident Energy (MeV)

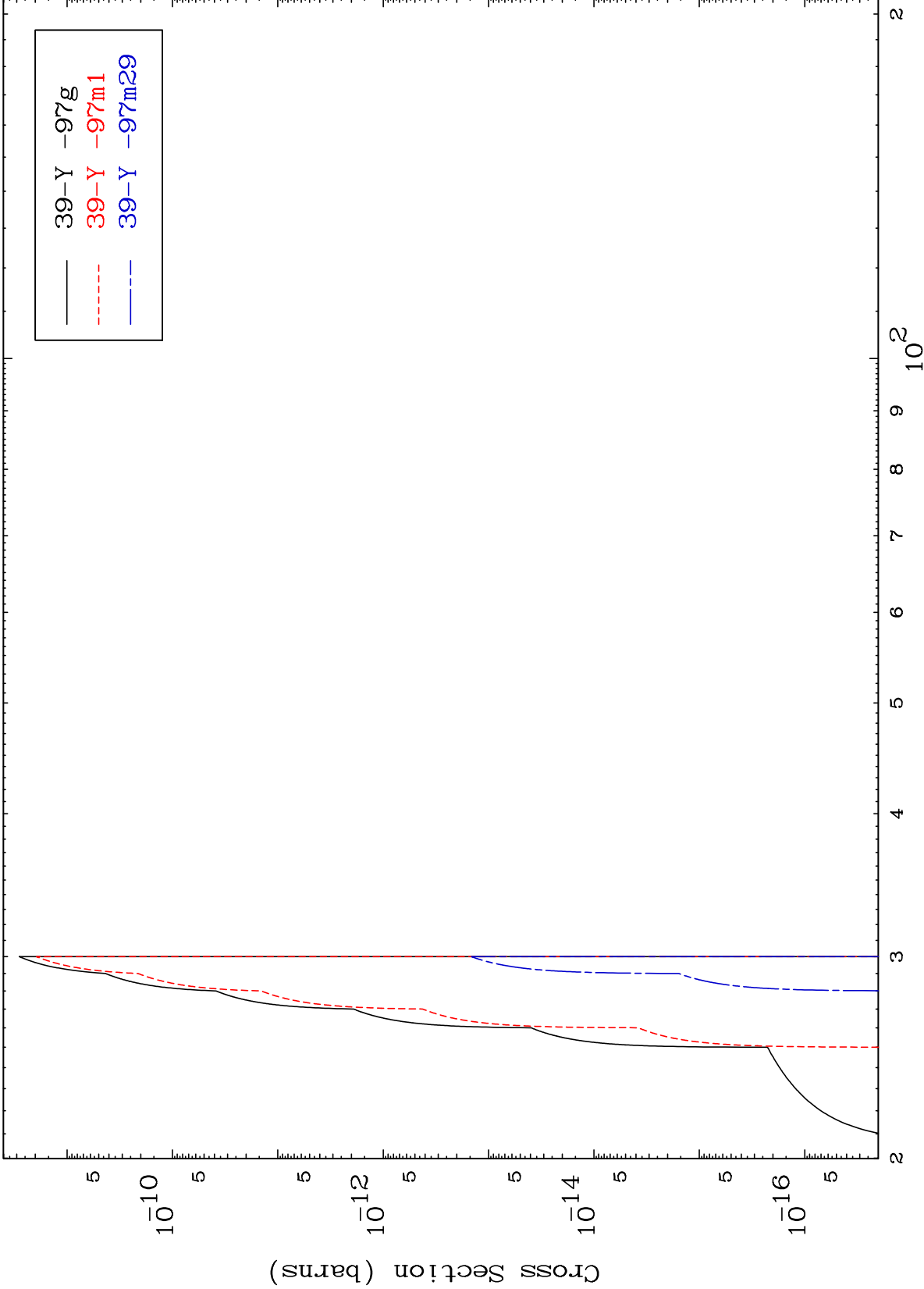
41-Nb-99m

MAT 4144

(n,2n) p

41-Nb-99m

Radionuclide Production Cross Section



23

Incident Energy (MeV)

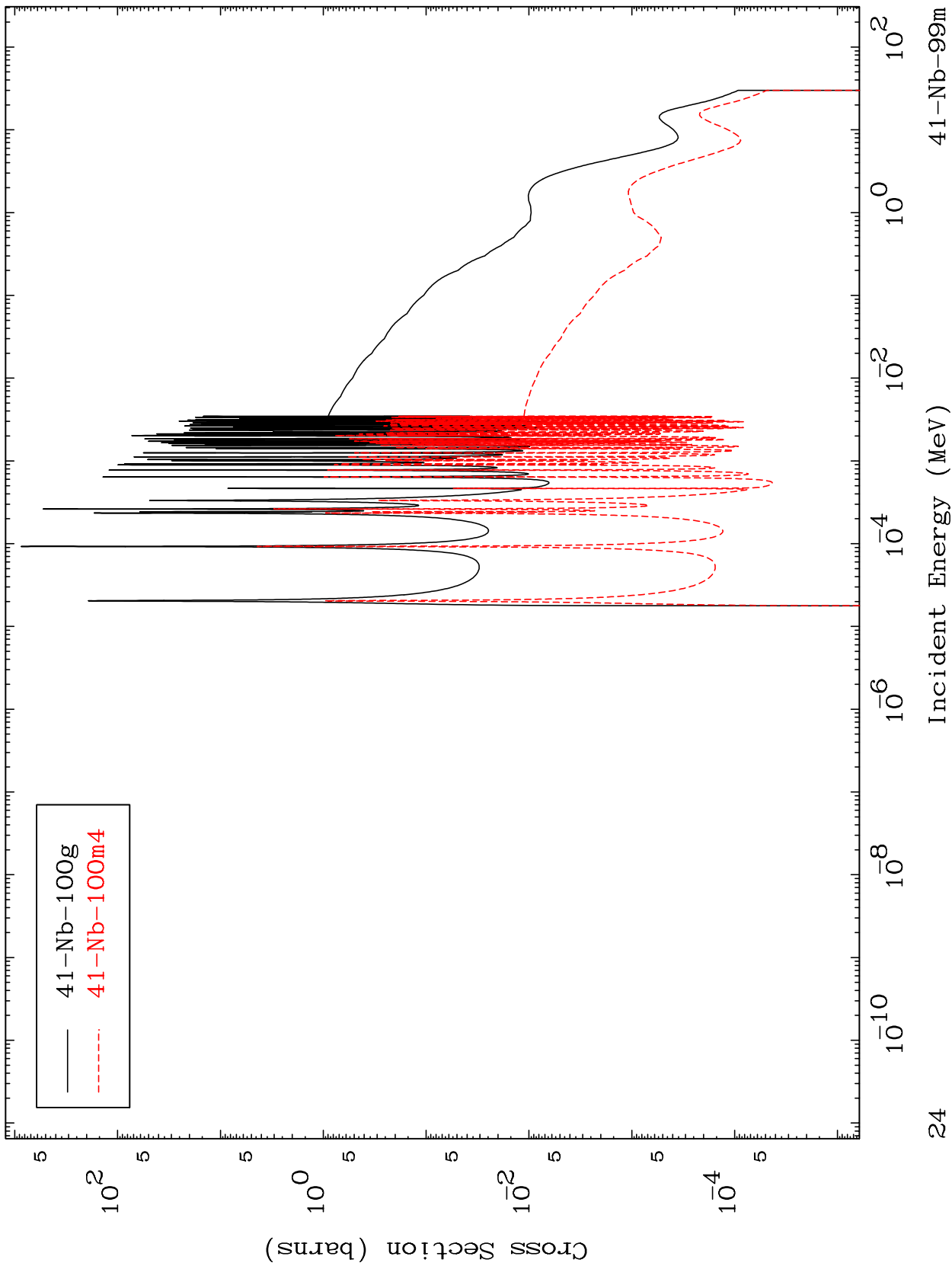
41-Nb-99m



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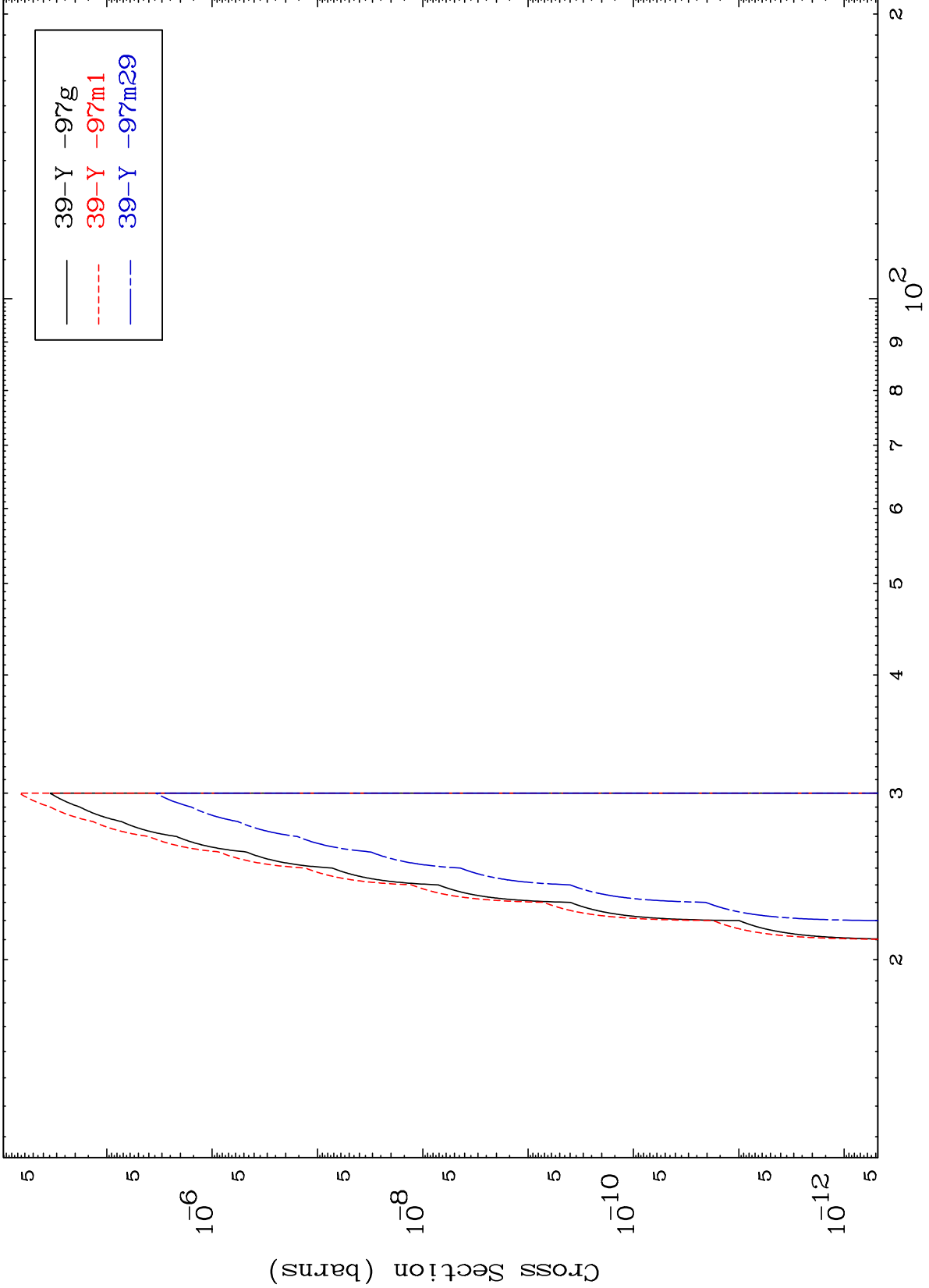
41-Nb-99m

(n,  $\gamma$ )  
Radionuclide Production Cross Section



— 41-Nb-100g  
- - - 41-Nb-100m4

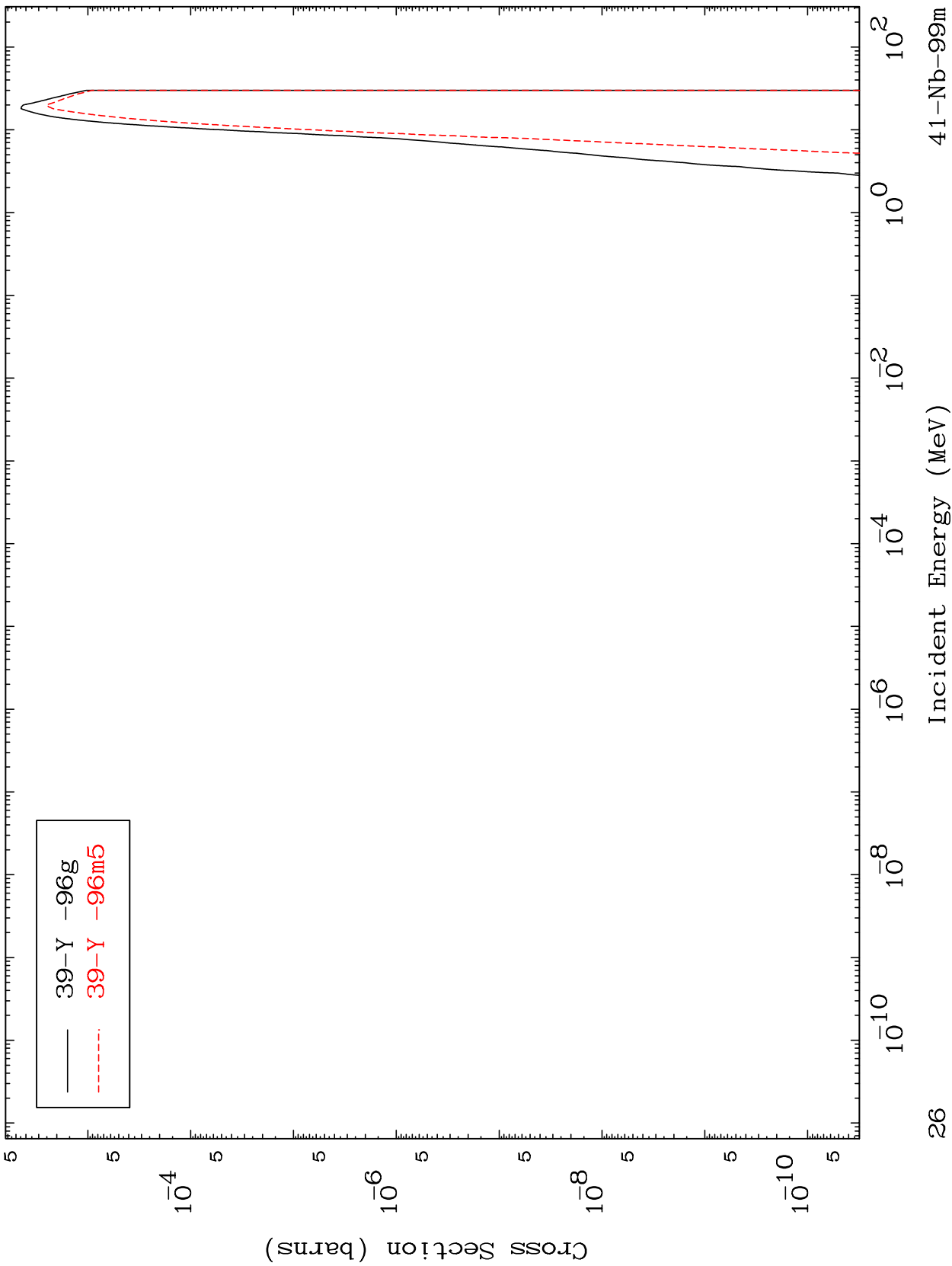
Radionuclide Production Cross Section



MAT 4144

41-Nb-99m

Radionuclide Production Cross Section

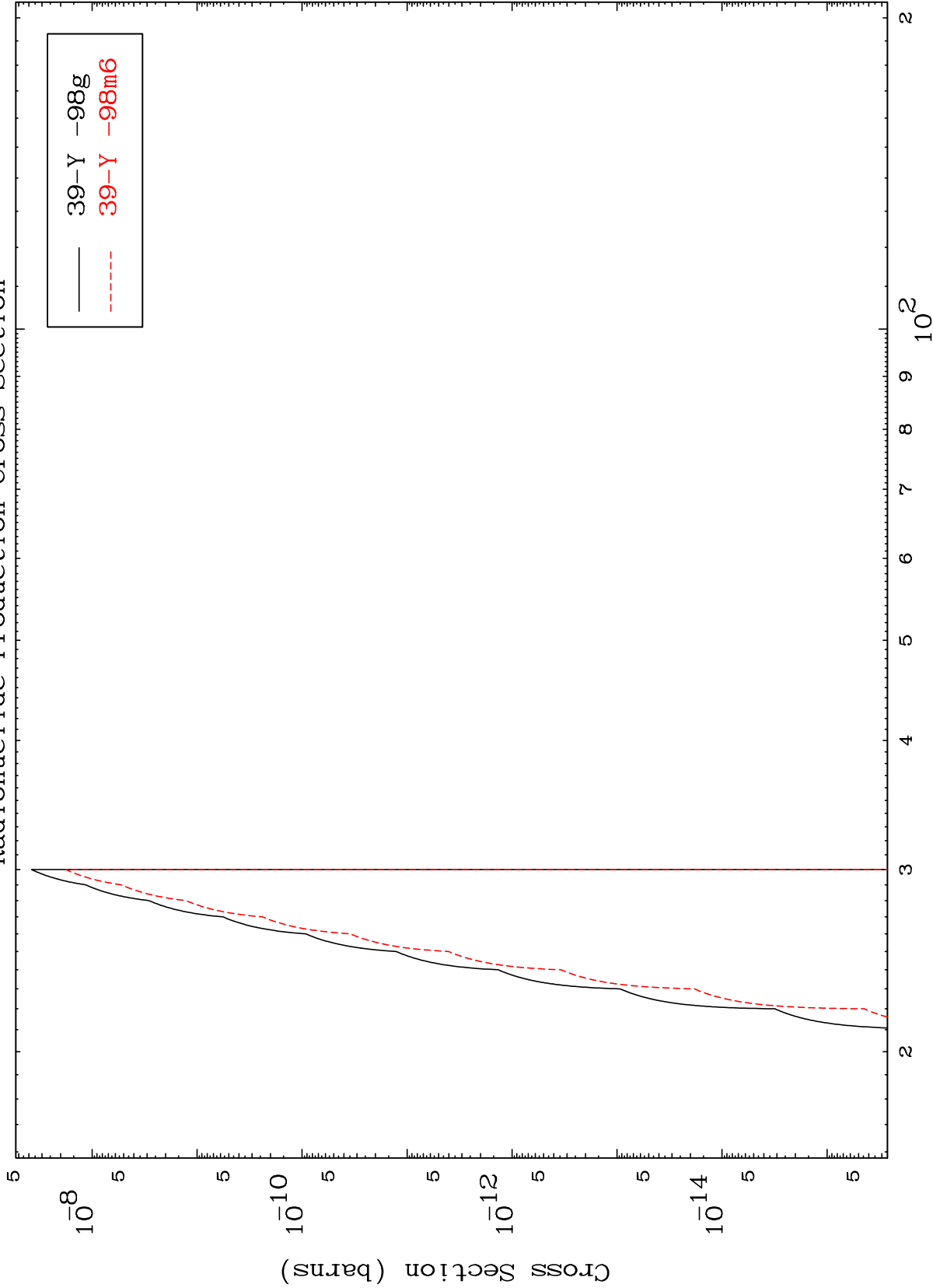


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(n,2p)

41-Nb-99m

Radionuclide Production Cross Section



27

Incident Energy (MeV)

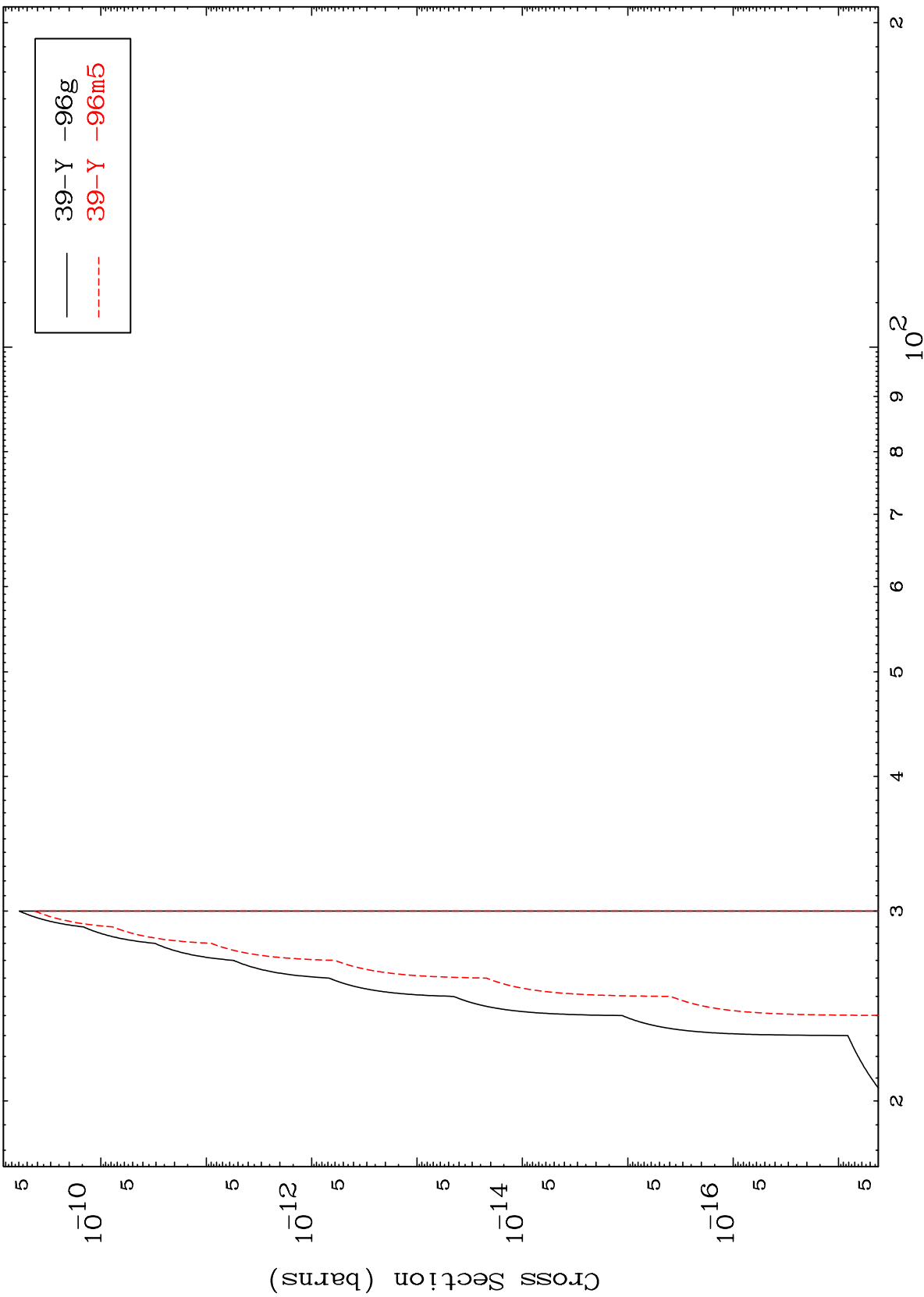
41-Nb-99m

MAT 4144

(n,p) t

41-Nb-99m

Radionuclide Production Cross Section



28

Incident Energy (MeV)

41-Nb-99m