

Program EVALPLOT
(Version 2021-1)

by

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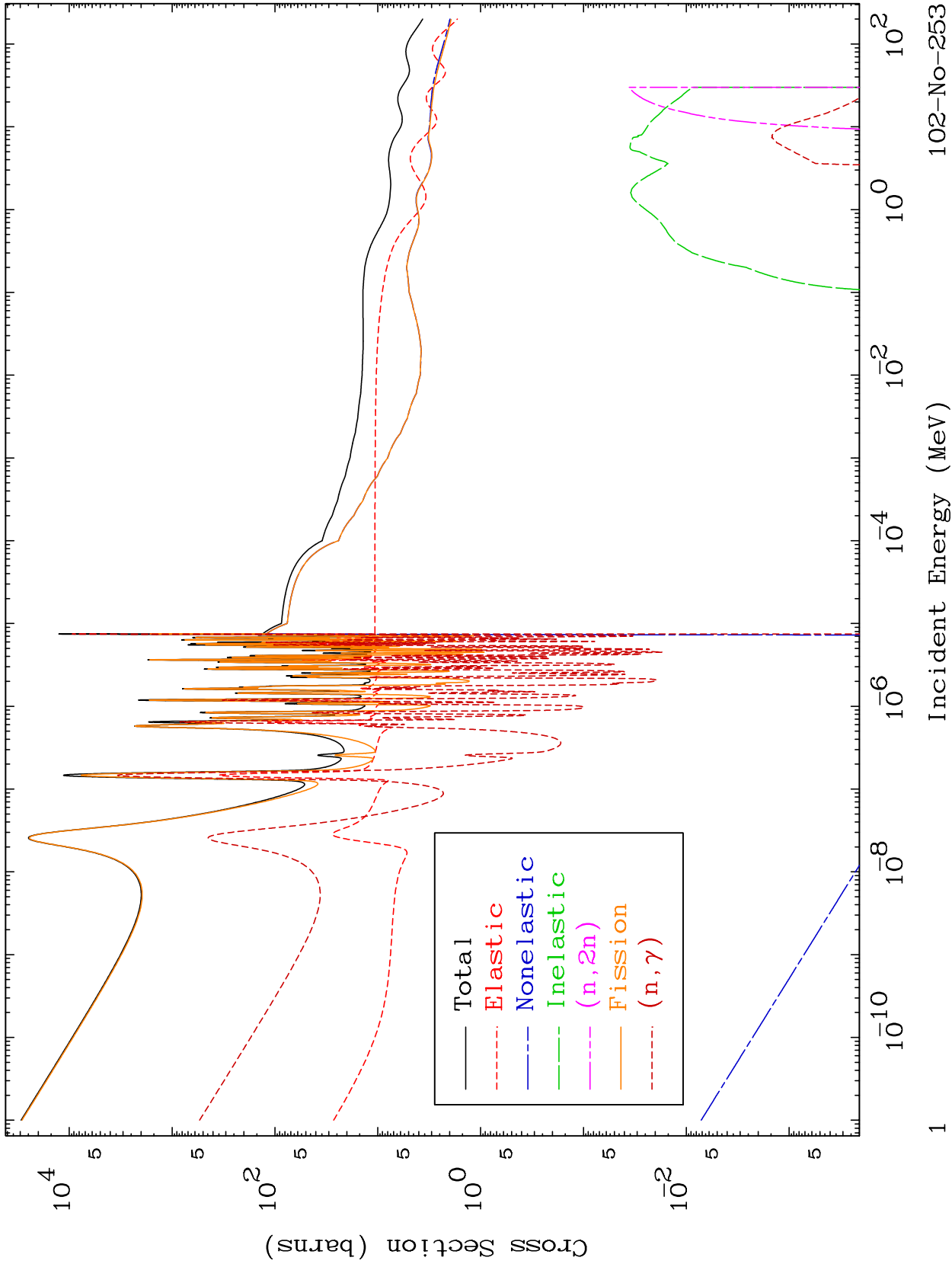
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Press Mouse Button to Start

MAT 9971

Neutron Major
293 Kelvin Cross Sections

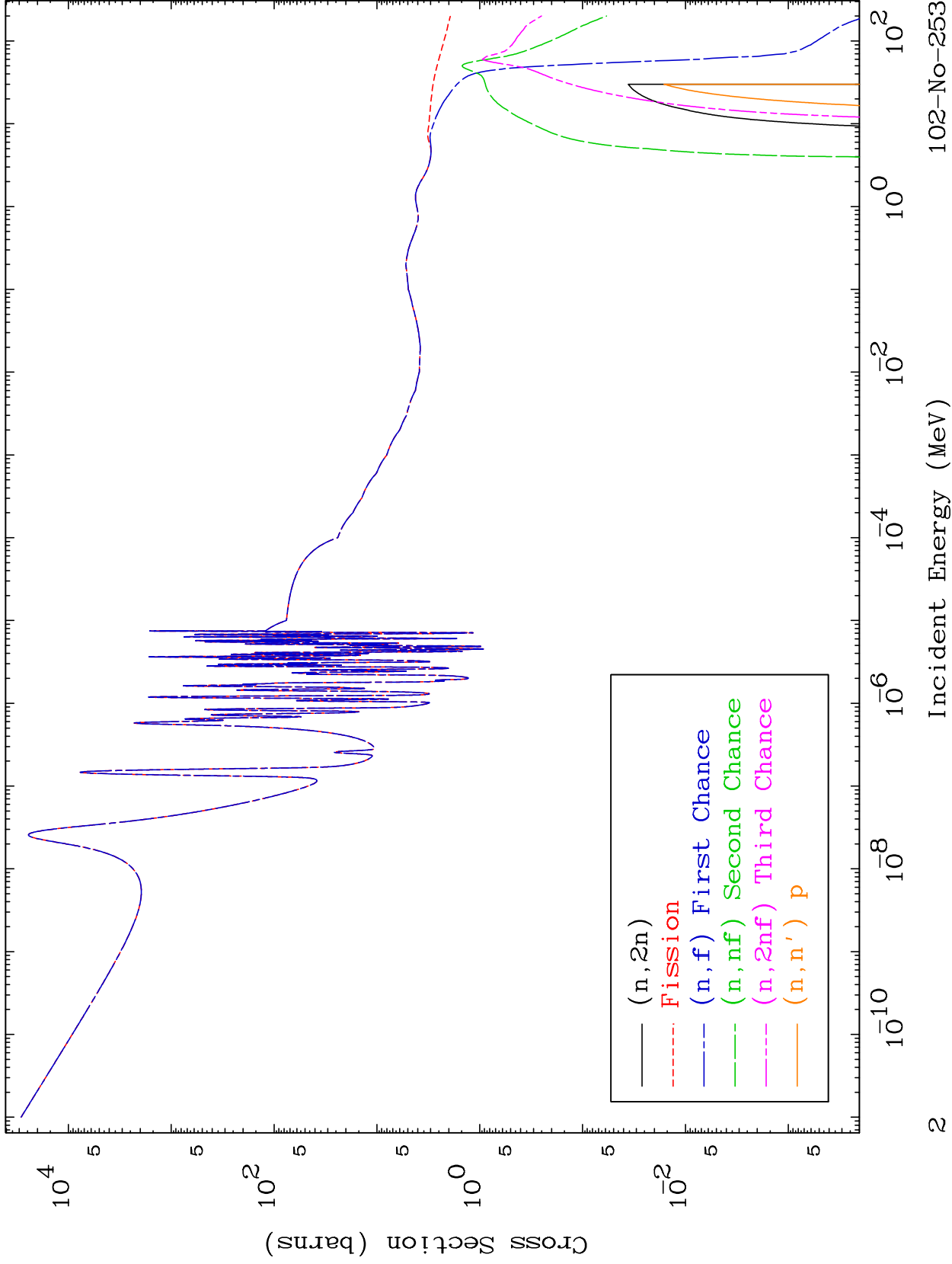
102-No-253



MAT 9971

Neutron Absorption
293 Kelvin Cross Sections

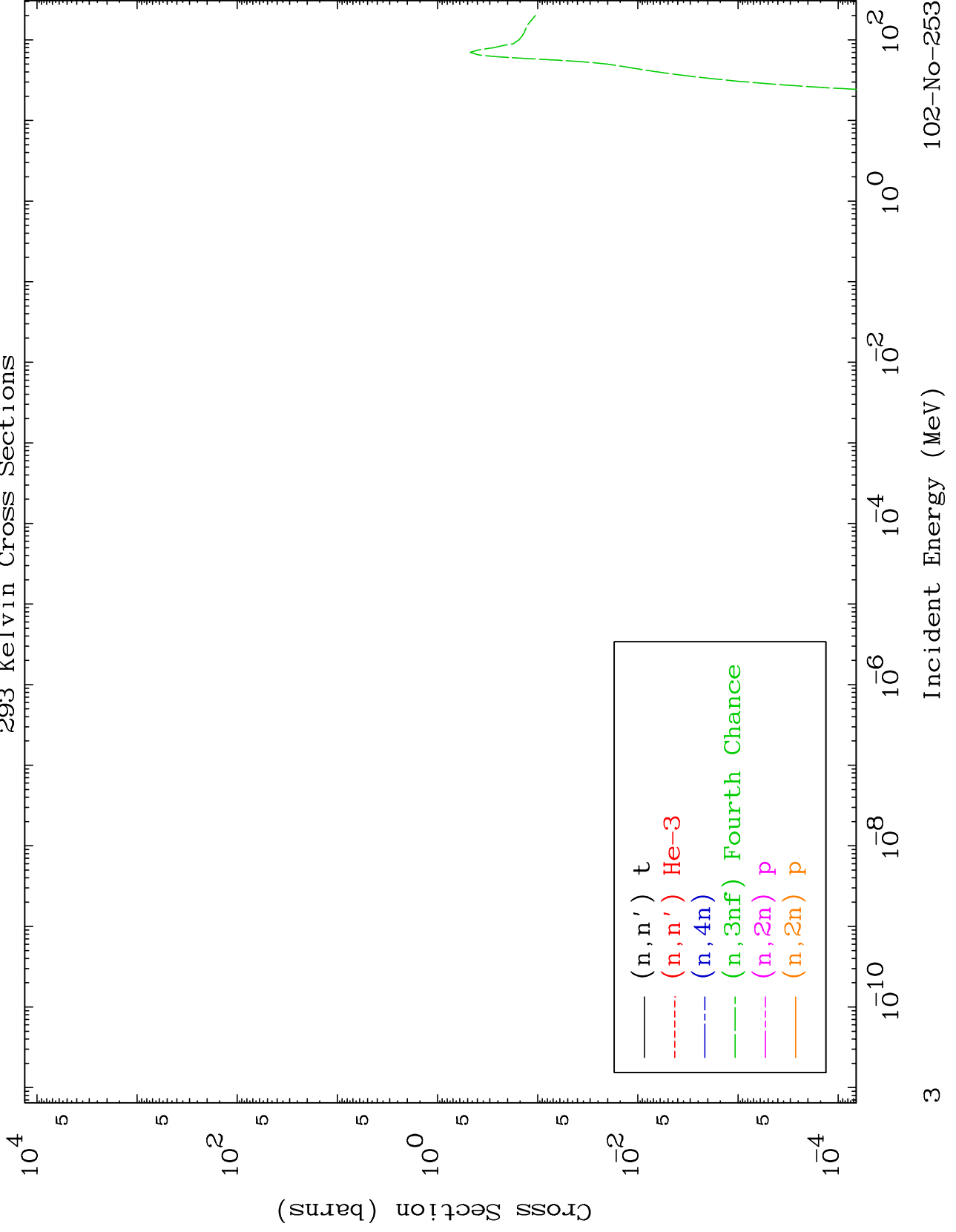
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MAT 9971

Neutron Absorption
293 Kelvin Cross Sections

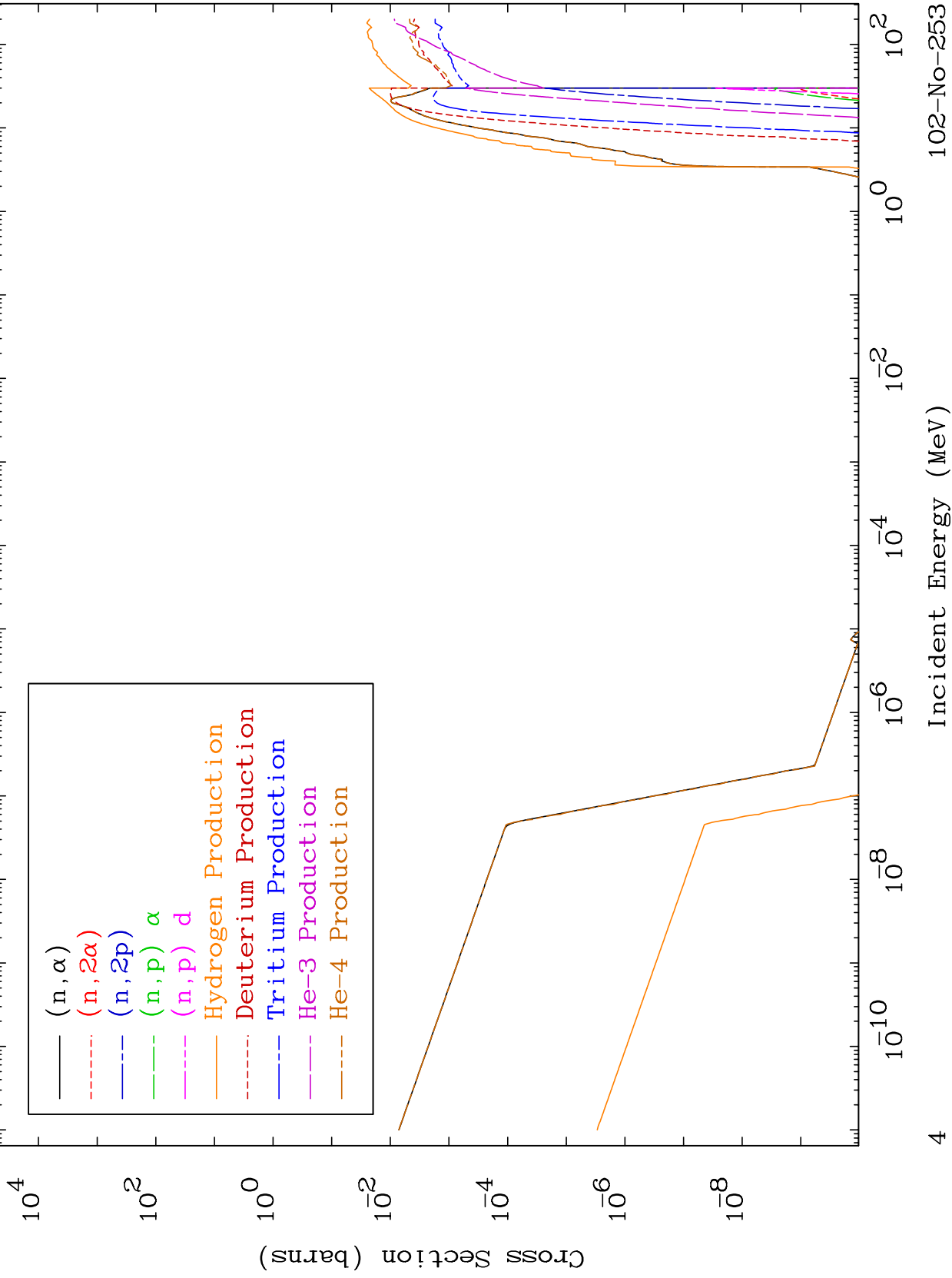
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MAT 9971

Neutron Absorption
293 Kelvin Cross Sections

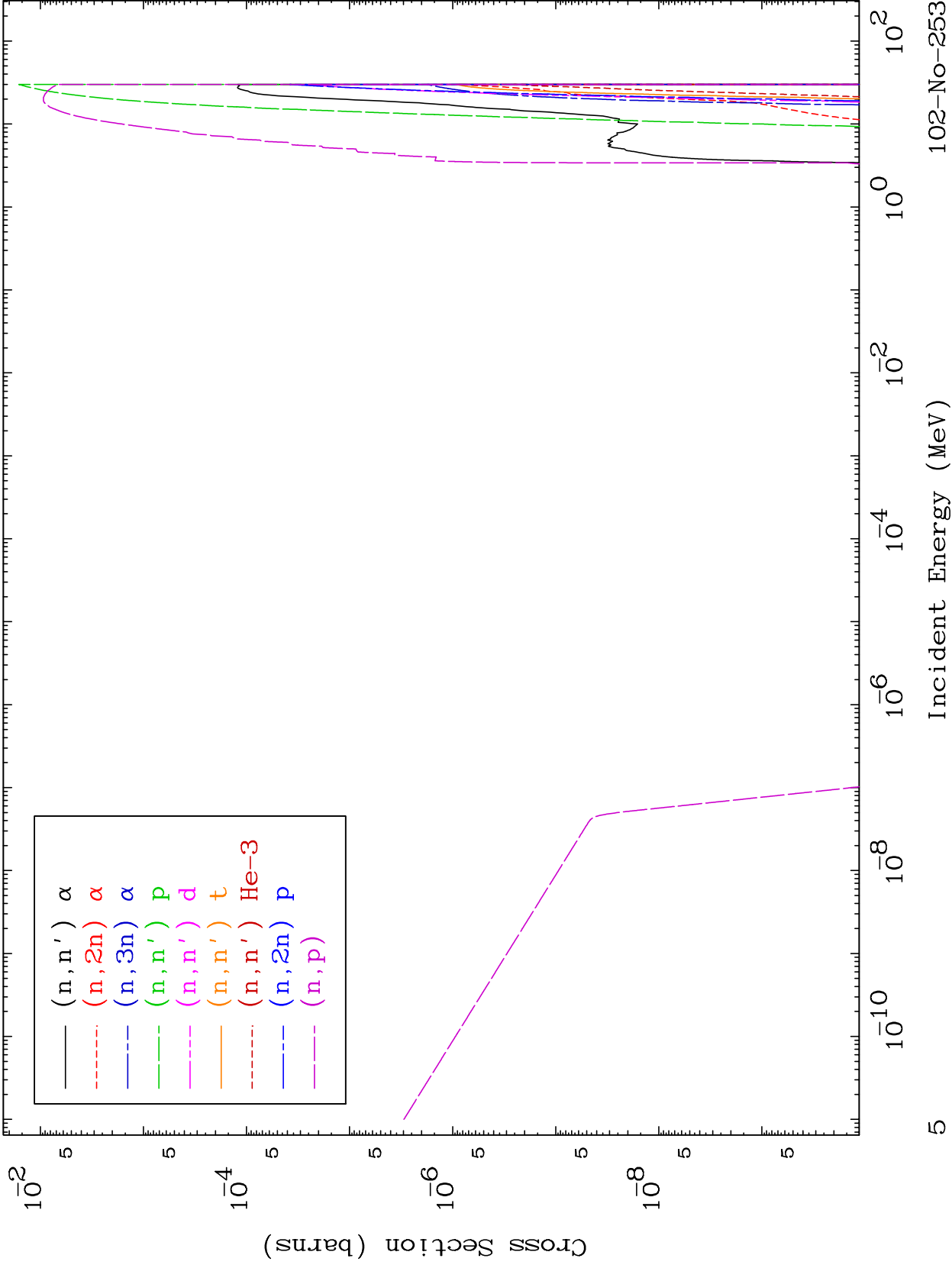
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MAT 9971

Charged Particle
293 Kelvin Cross Sections

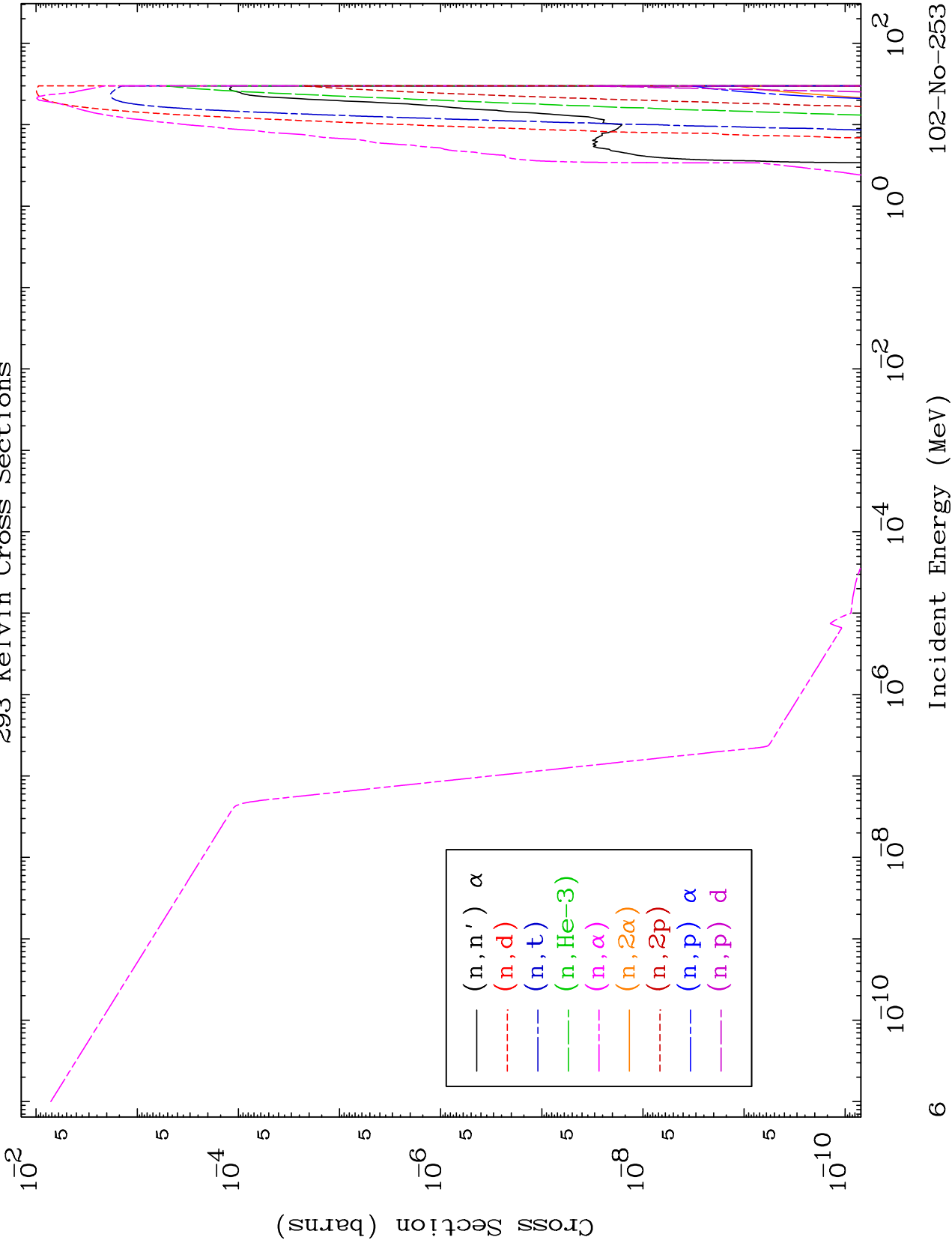
102-No-253



MAT 9971

Charged Particle
293 Kelvin Cross Sections

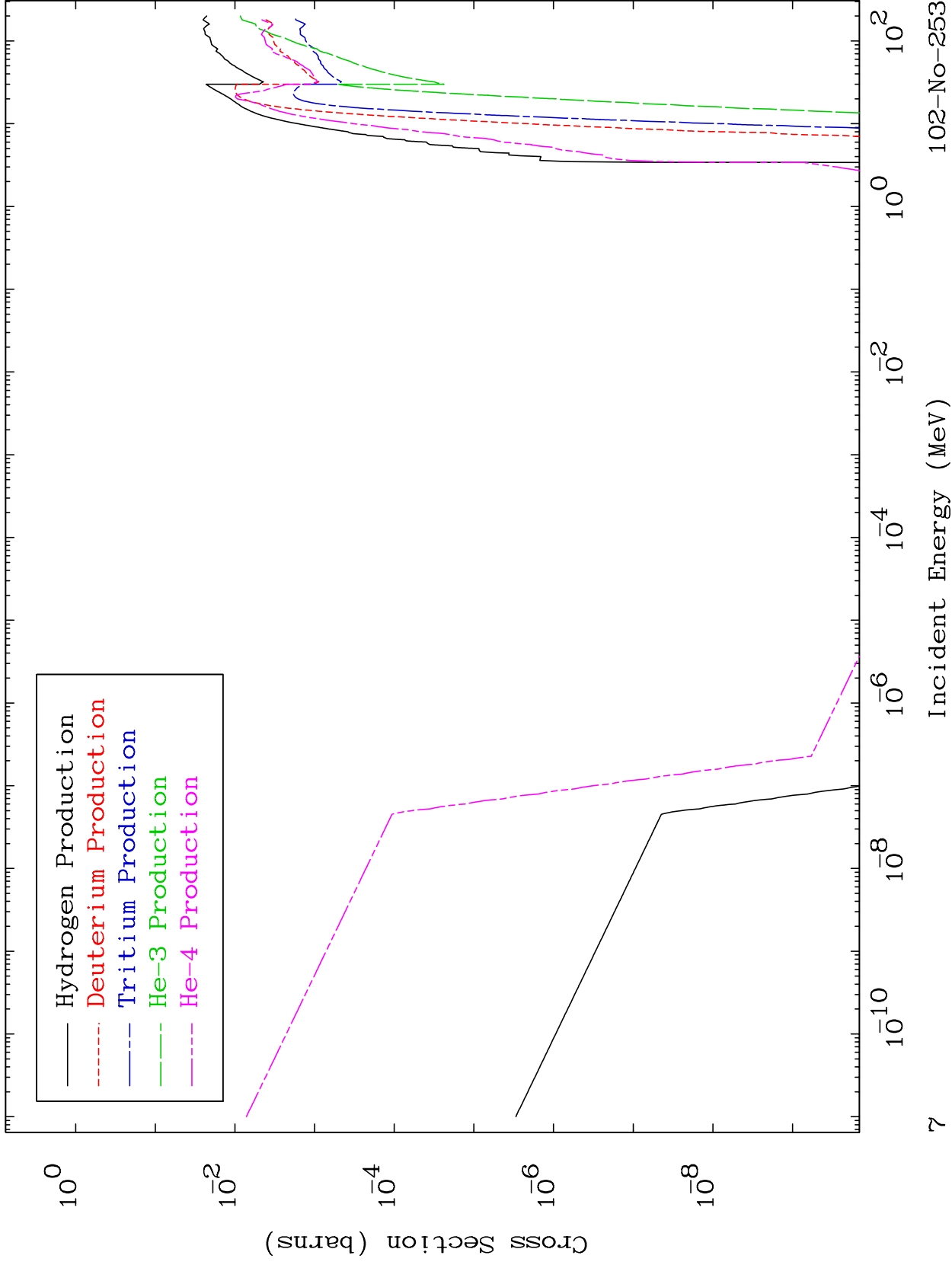
102-No-253



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Particle Production
293 Kelvin Cross Sections

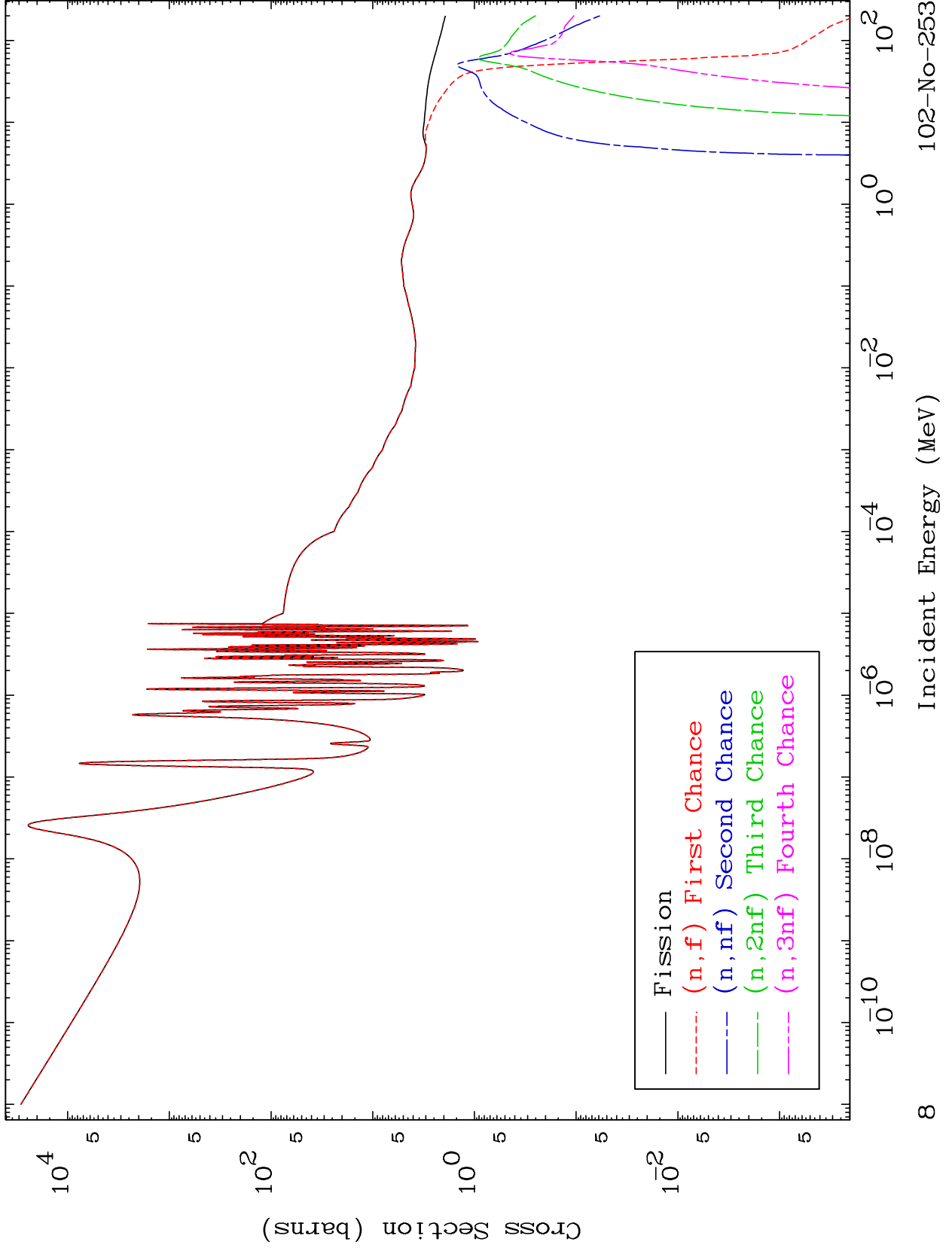
102-No-253



MAT 9971

Fission
293 Kelvin Cross Sections

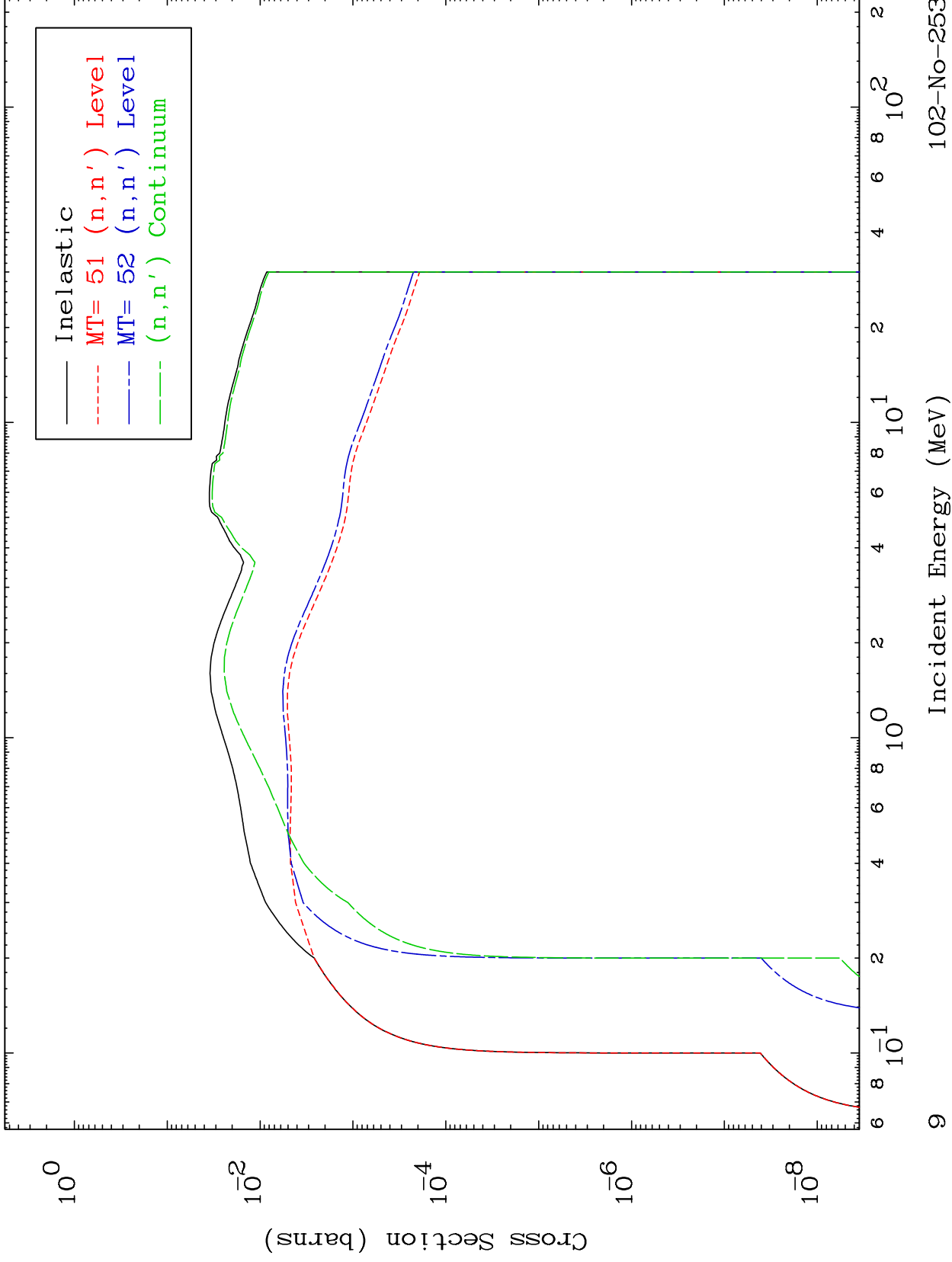
102-No-253



MAT 9971

(n,n') Levels
293 Kelvin Cross Sections

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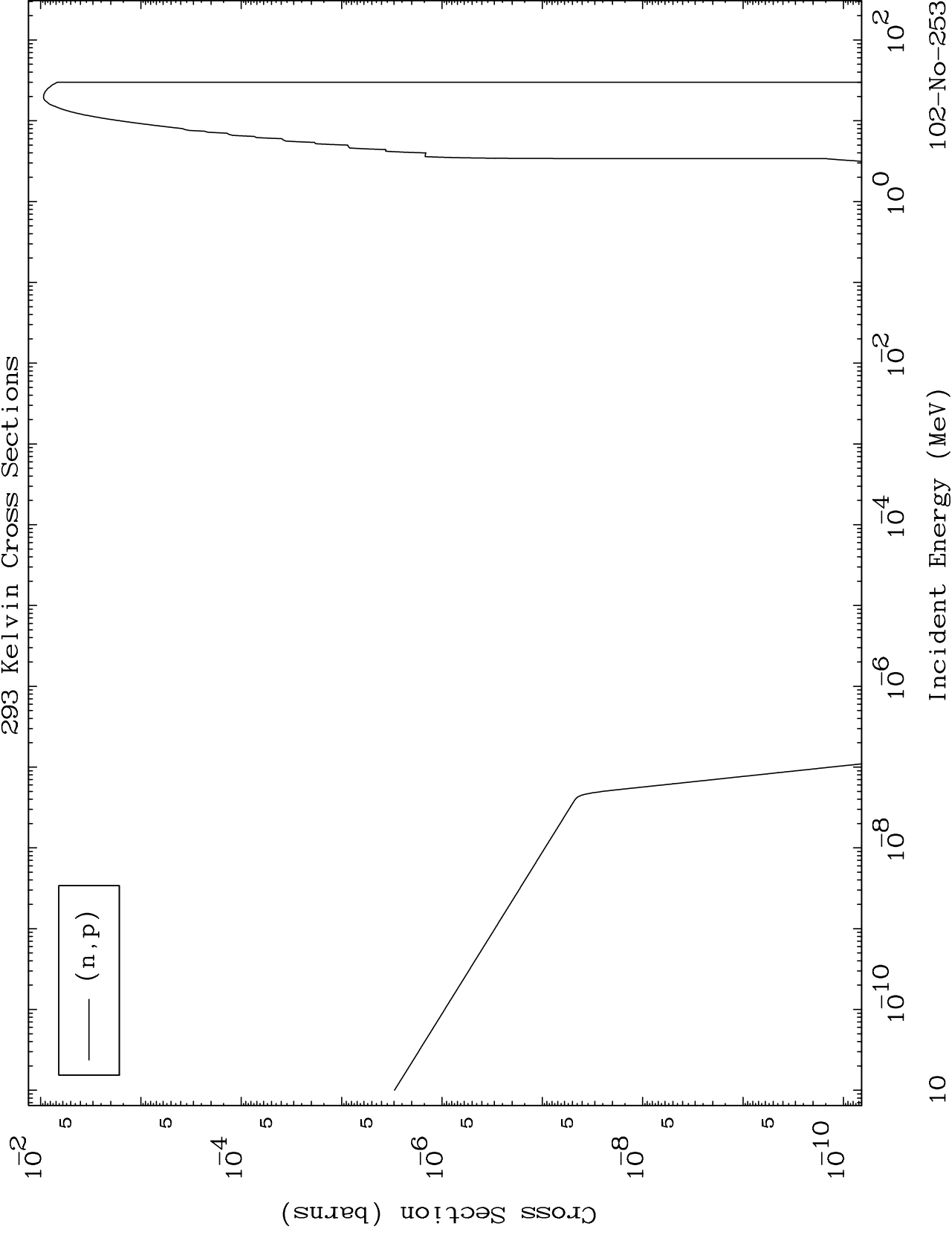
Incident Energy (MeV)

102-No-253

MAT 9971

(n,p) Levels
293 Kelvin Cross Sections

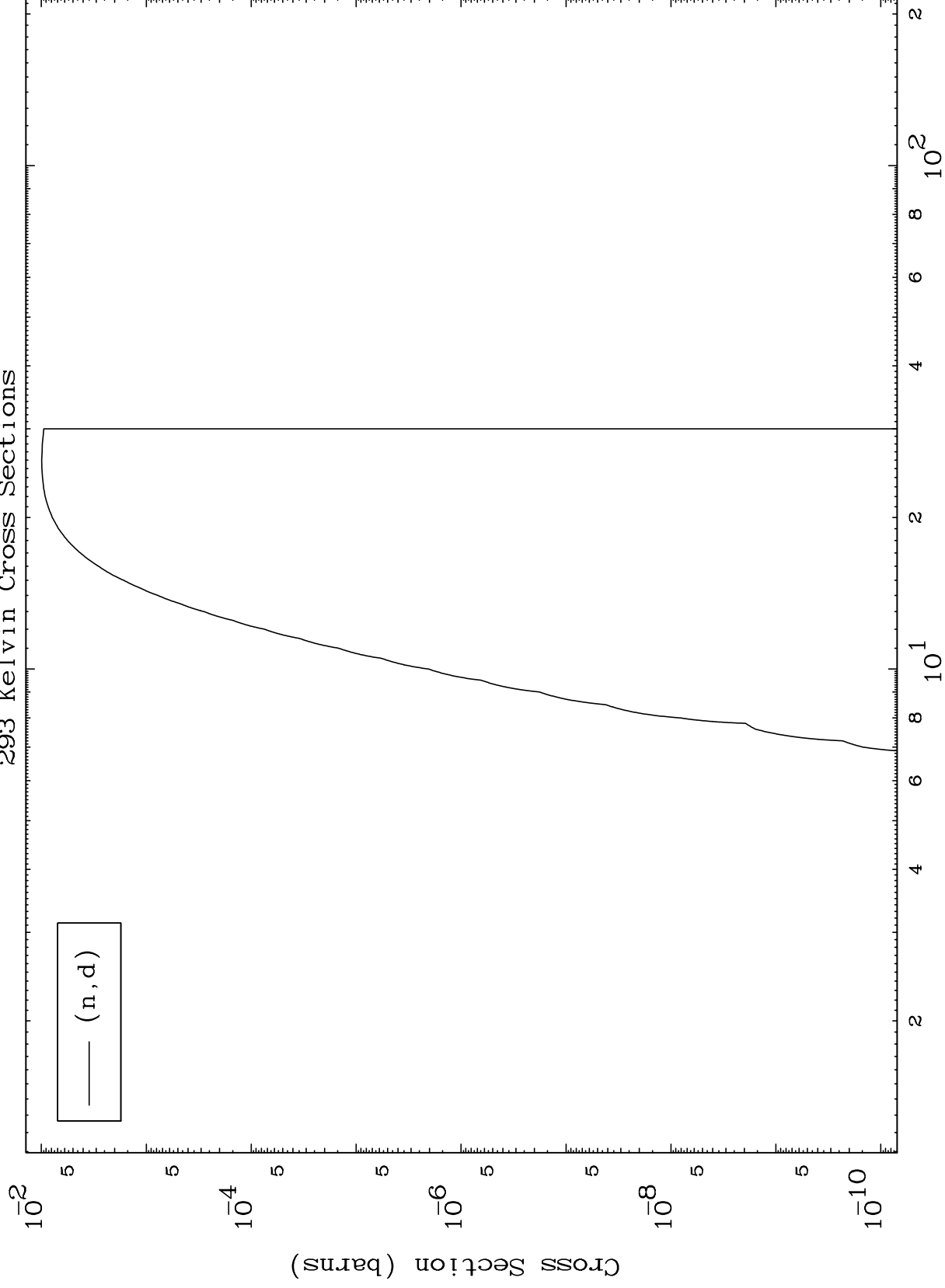
102-No-253



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(n,d) Levels
293 Kelvin Cross Sections

102-No-253



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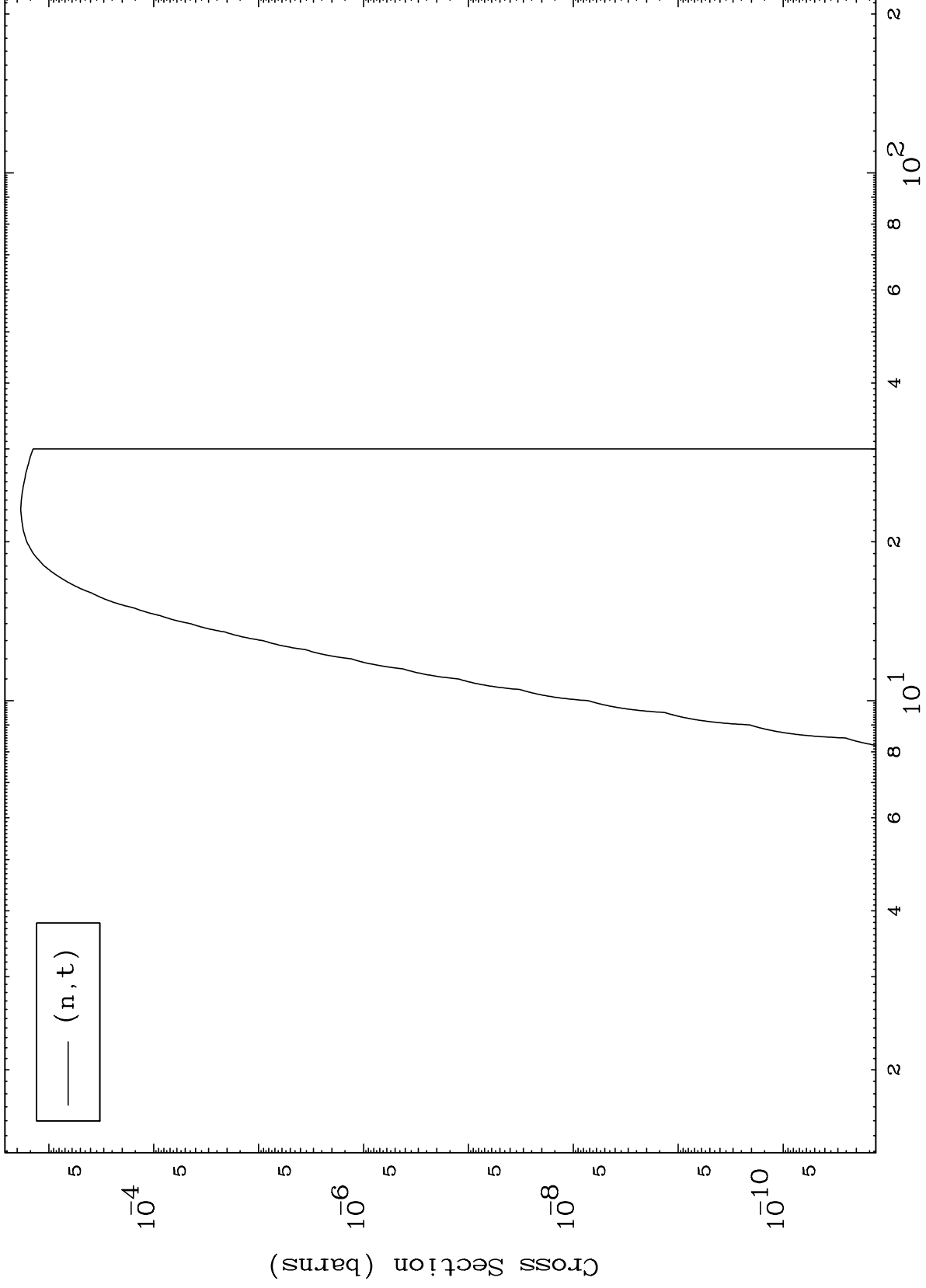
Incident Energy (MeV)

102-No-253

MAT 9971

(n,t) Levels
293 Kelvin Cross Sections

102-No-253



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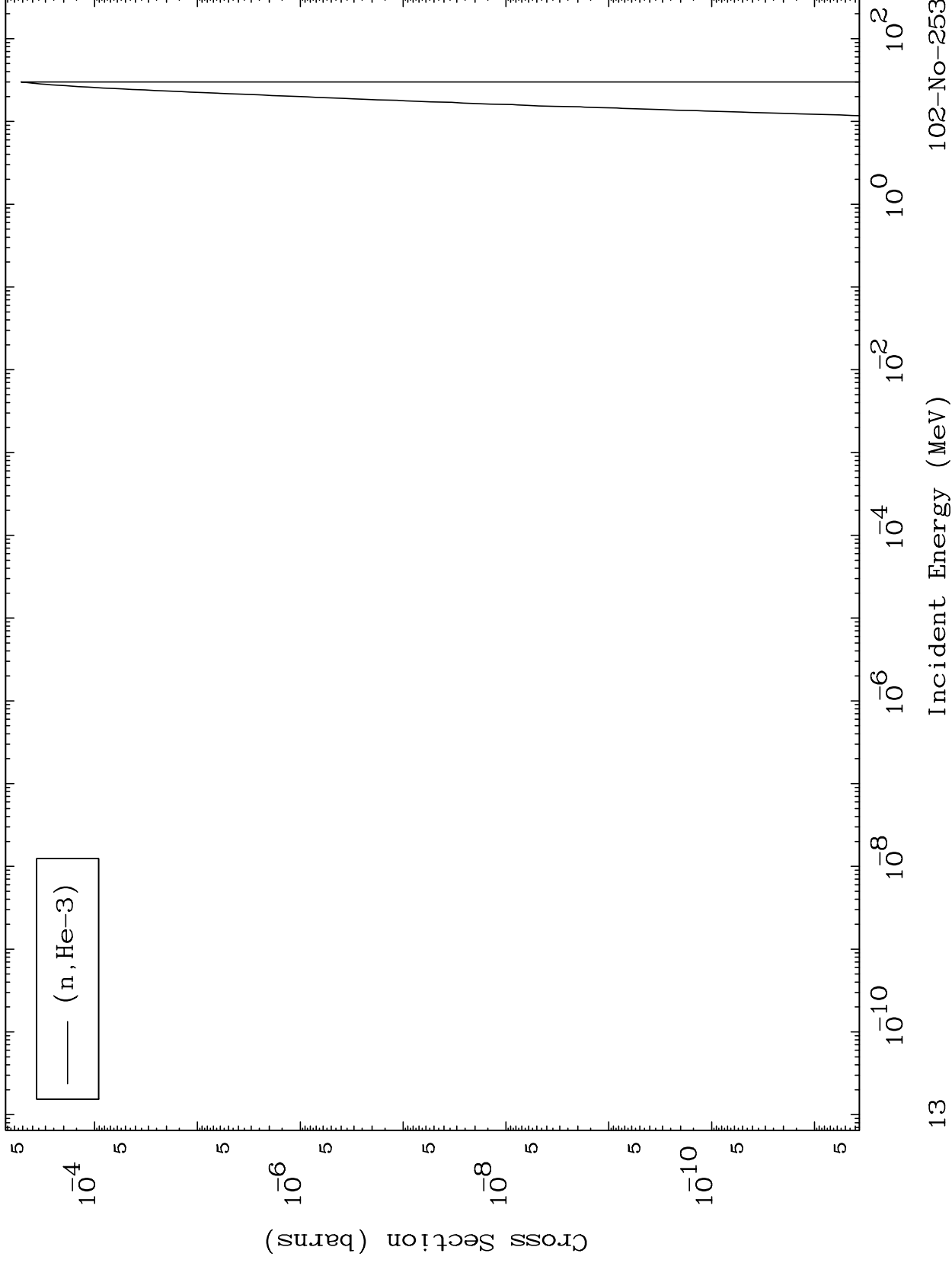
Incident Energy (MeV)

102-No-253

MAT 9971

(n,He3) Levels
293 Kelvin Cross Sections

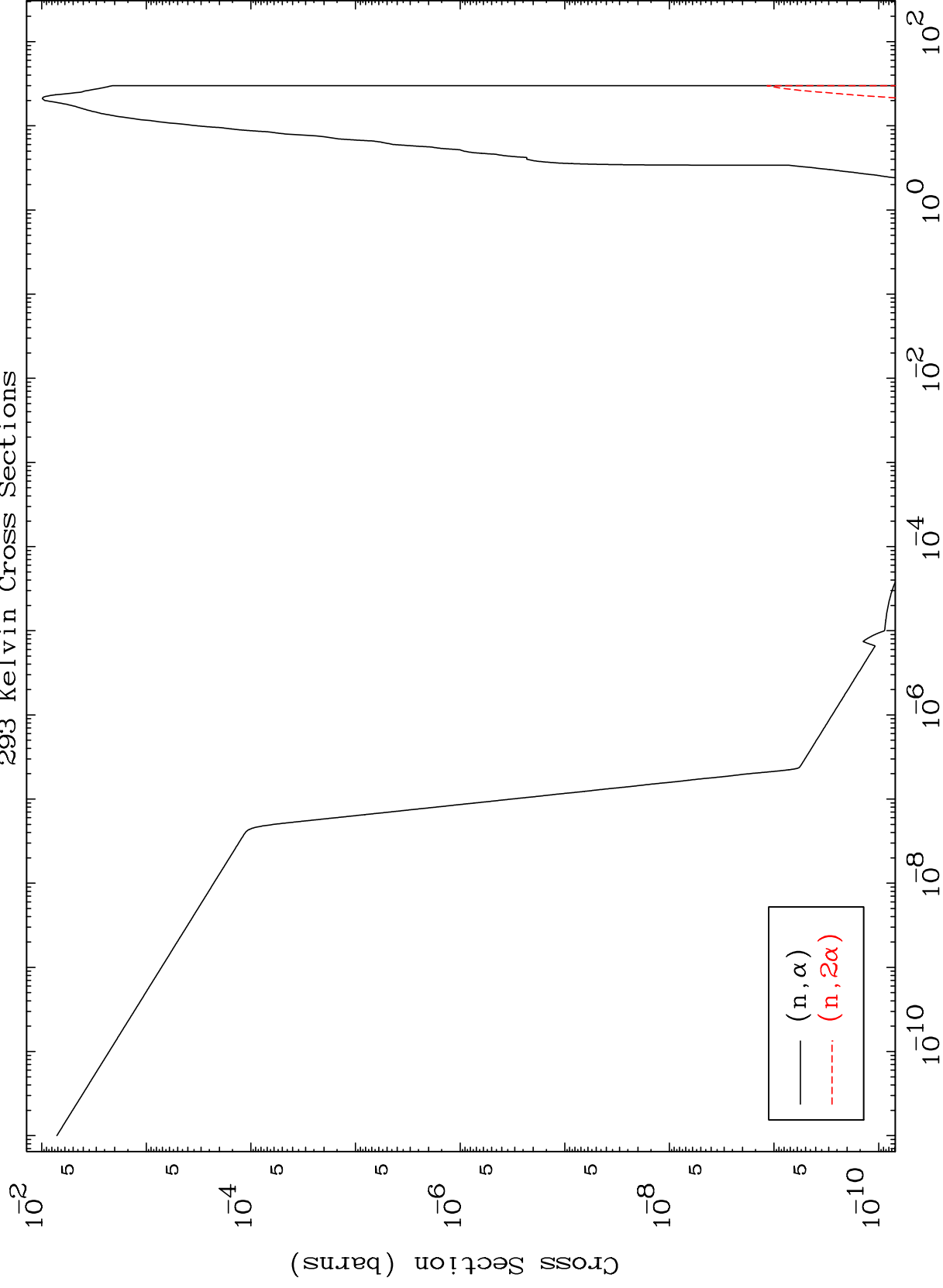
102-No-253



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(n, α) Levels
293 Kelvin Cross Sections

102-No-253



Incident Energy (MeV)

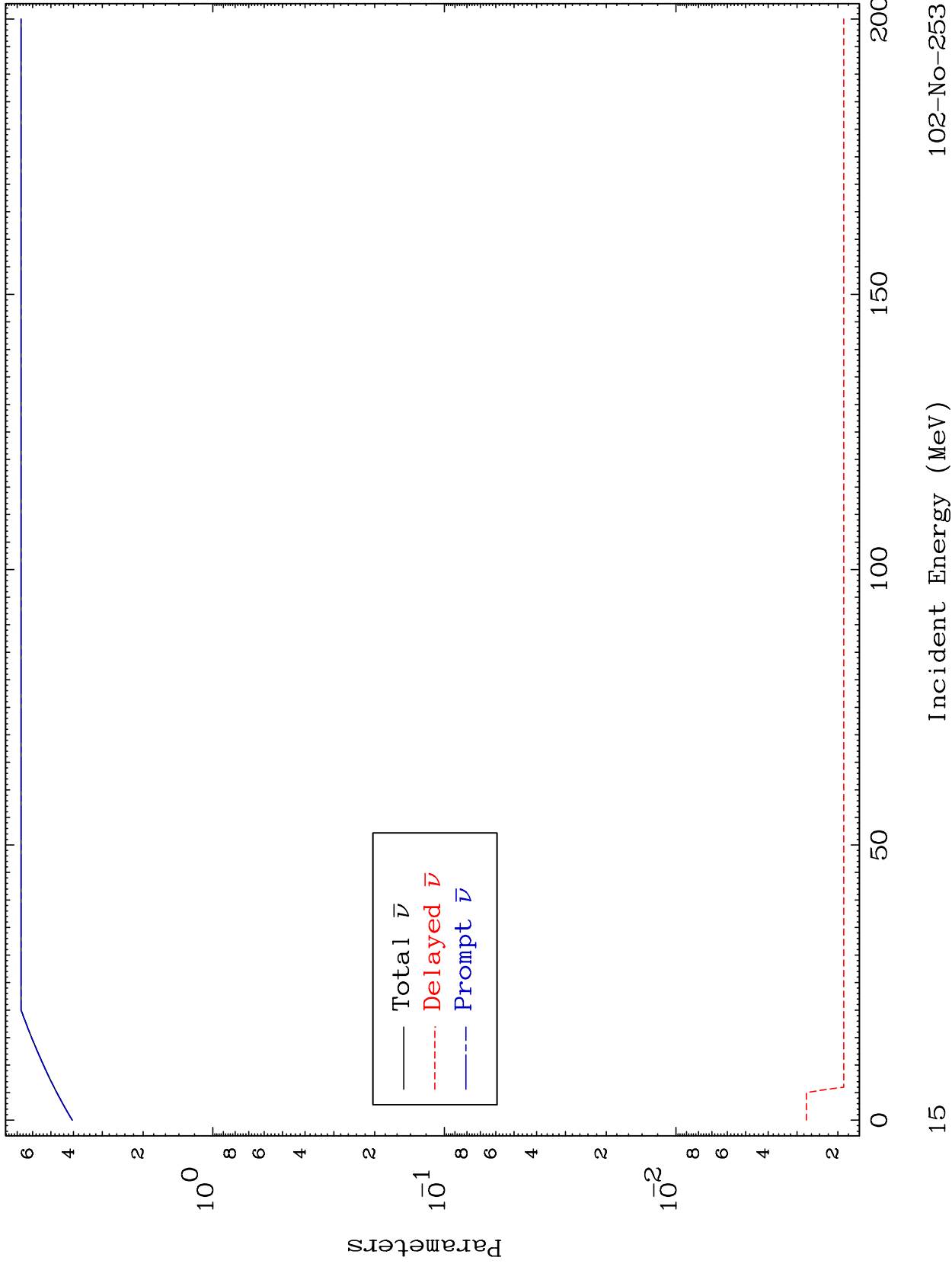
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Energy Release
Parameters

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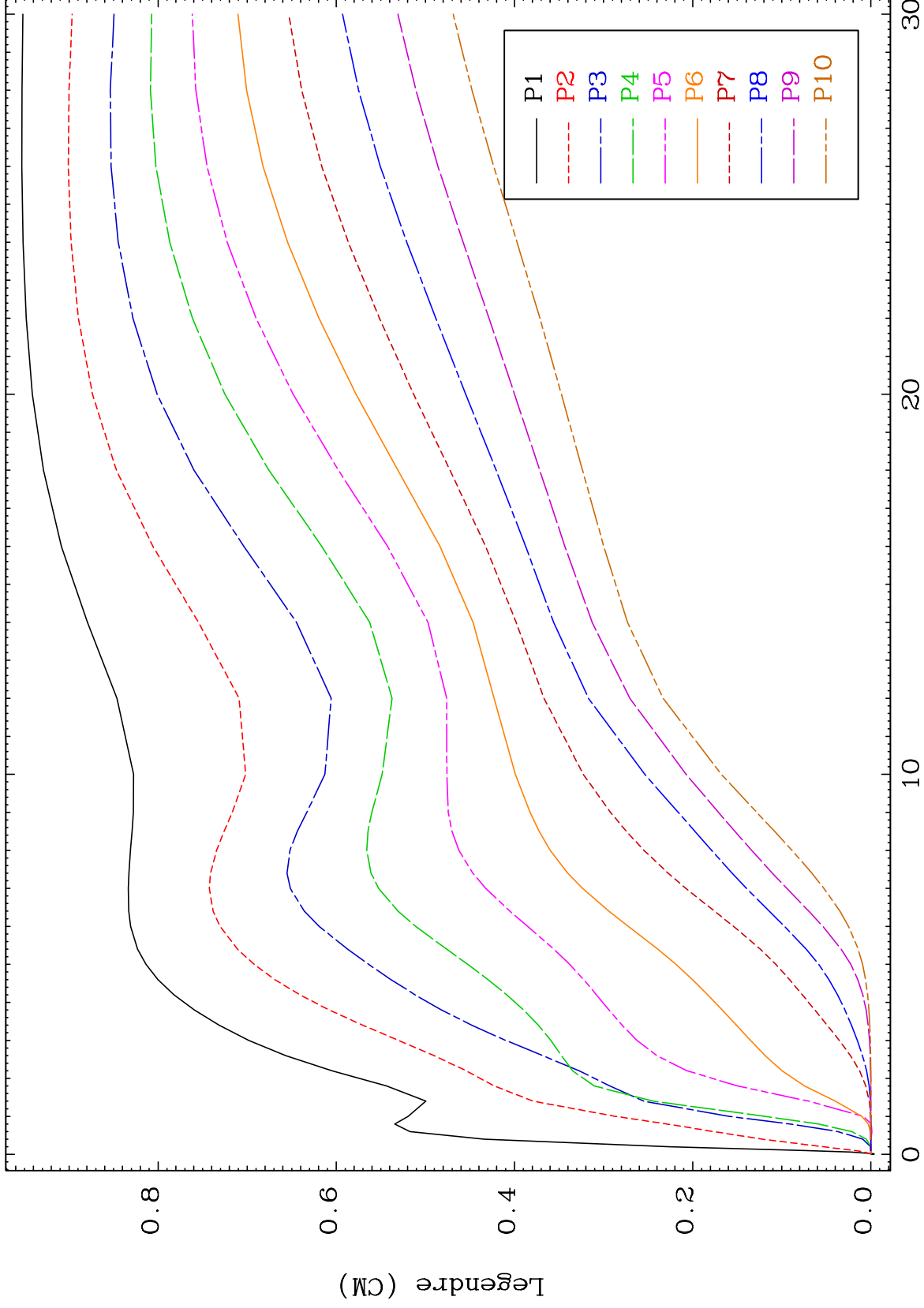
Incident Energy (MeV)

102-No-253

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Elastic Legendre Coefficients

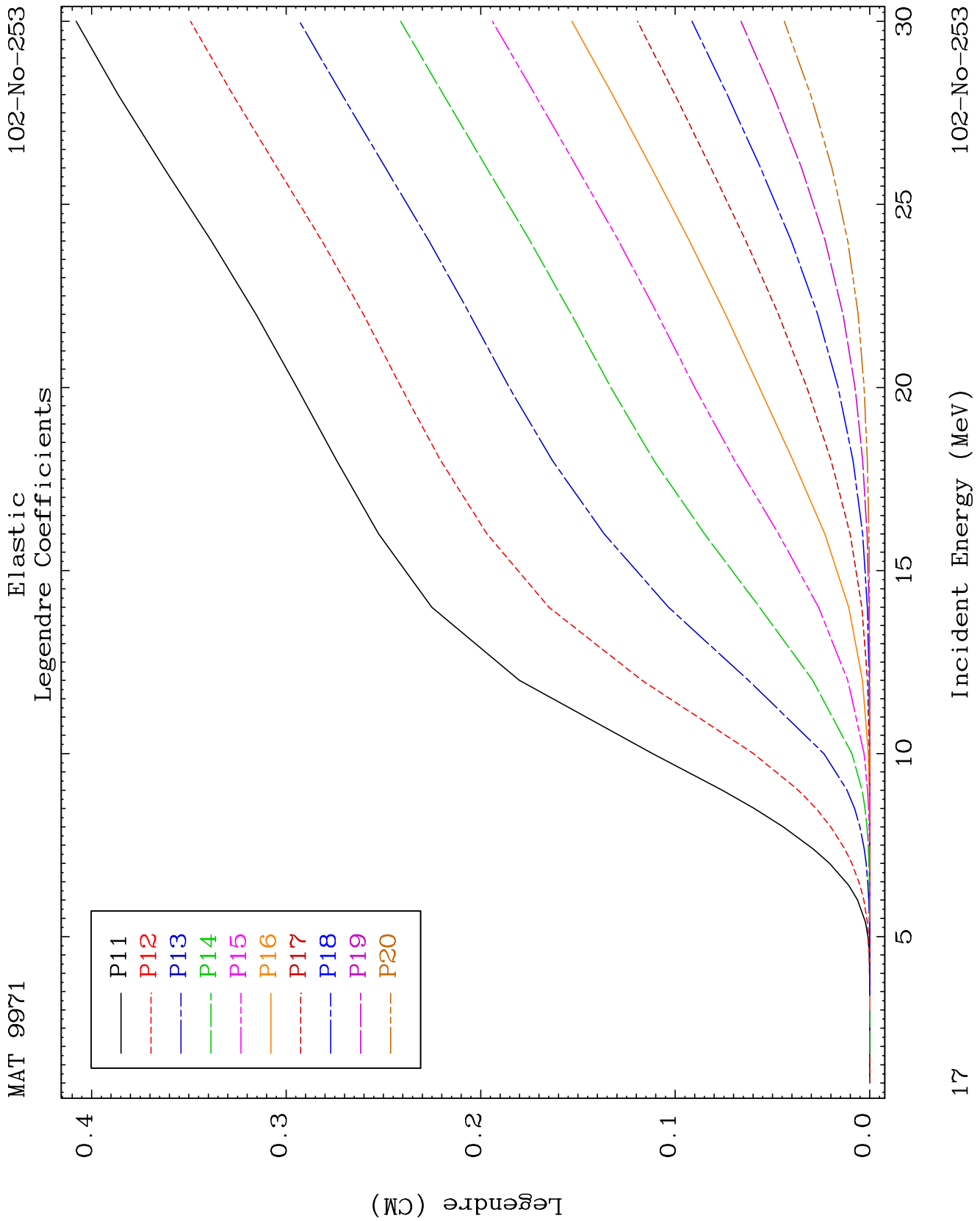
102-No-253



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Incident Energy (MeV)

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Elastic Legendre Coefficients

102-No-253

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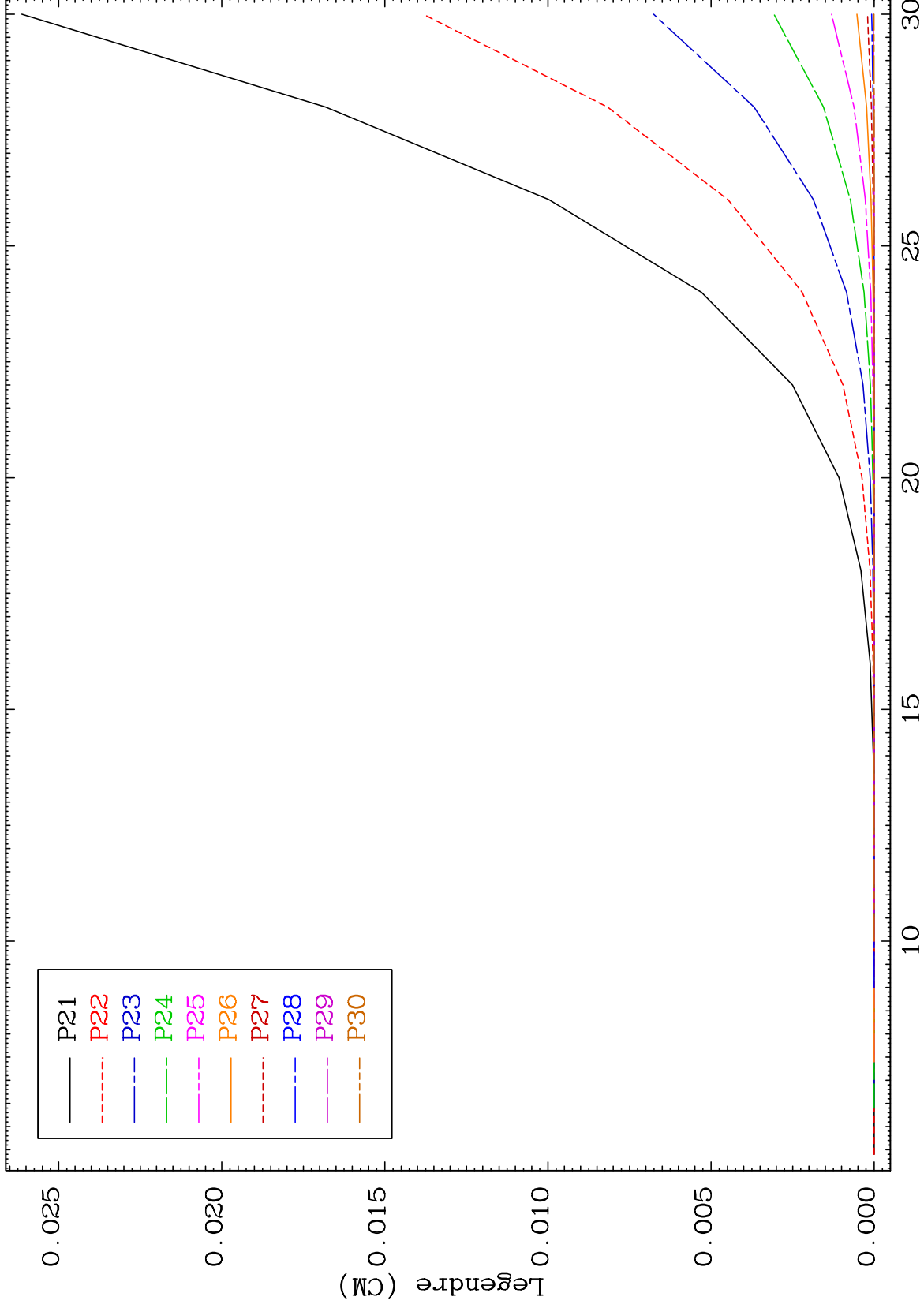
Incident Energy (MeV)

102-No-253

MAT 9971

Elastic Legendre Coefficients

102-No-253



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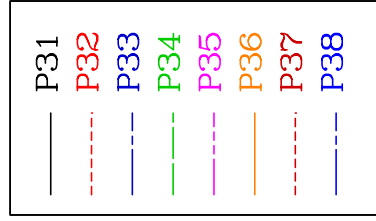
Incident Energy (MeV)

102-No-253

MAT 9971

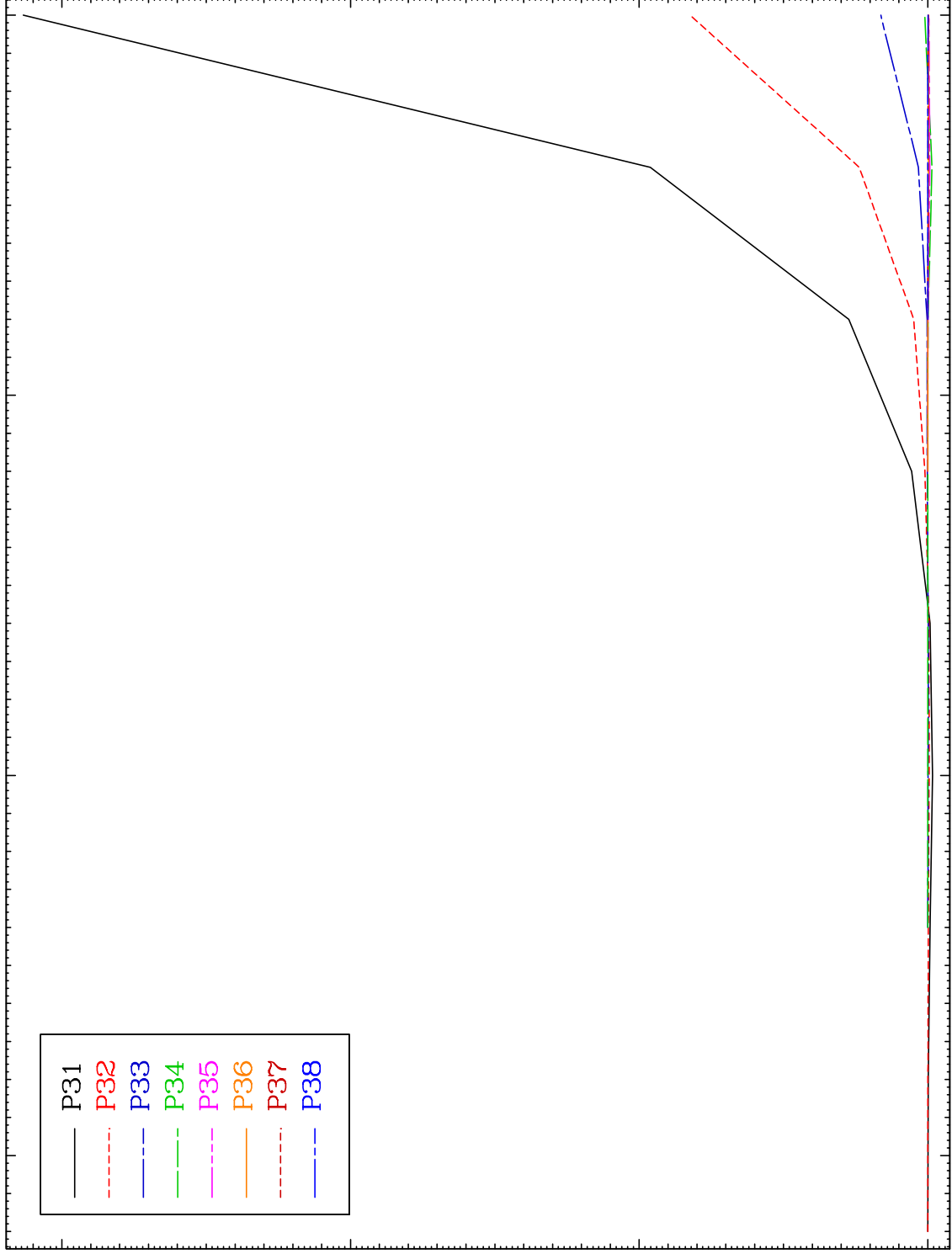
Elastic Legendre Coefficients

102-No-253



$\times 10^{-68}$

Legendre (CM)

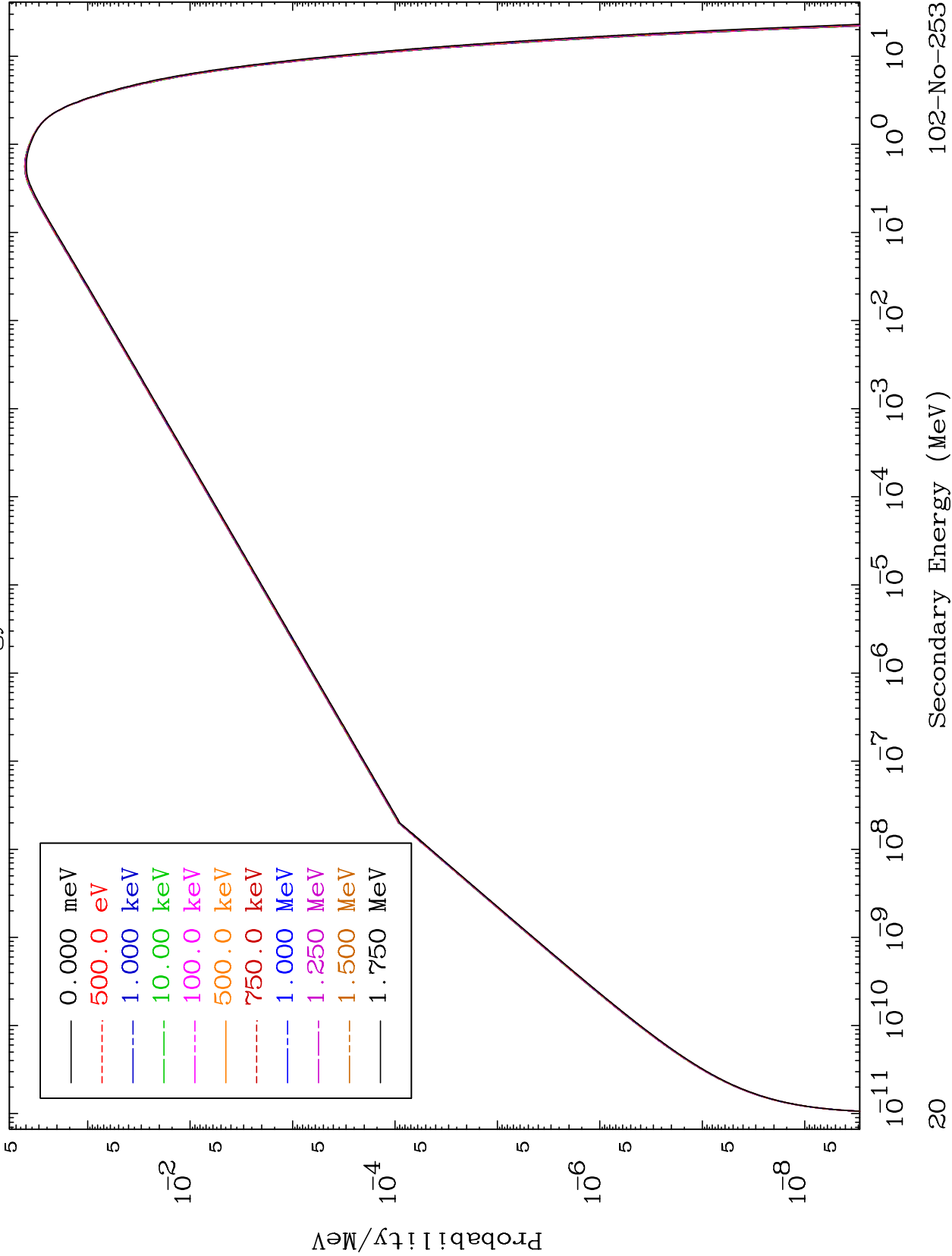


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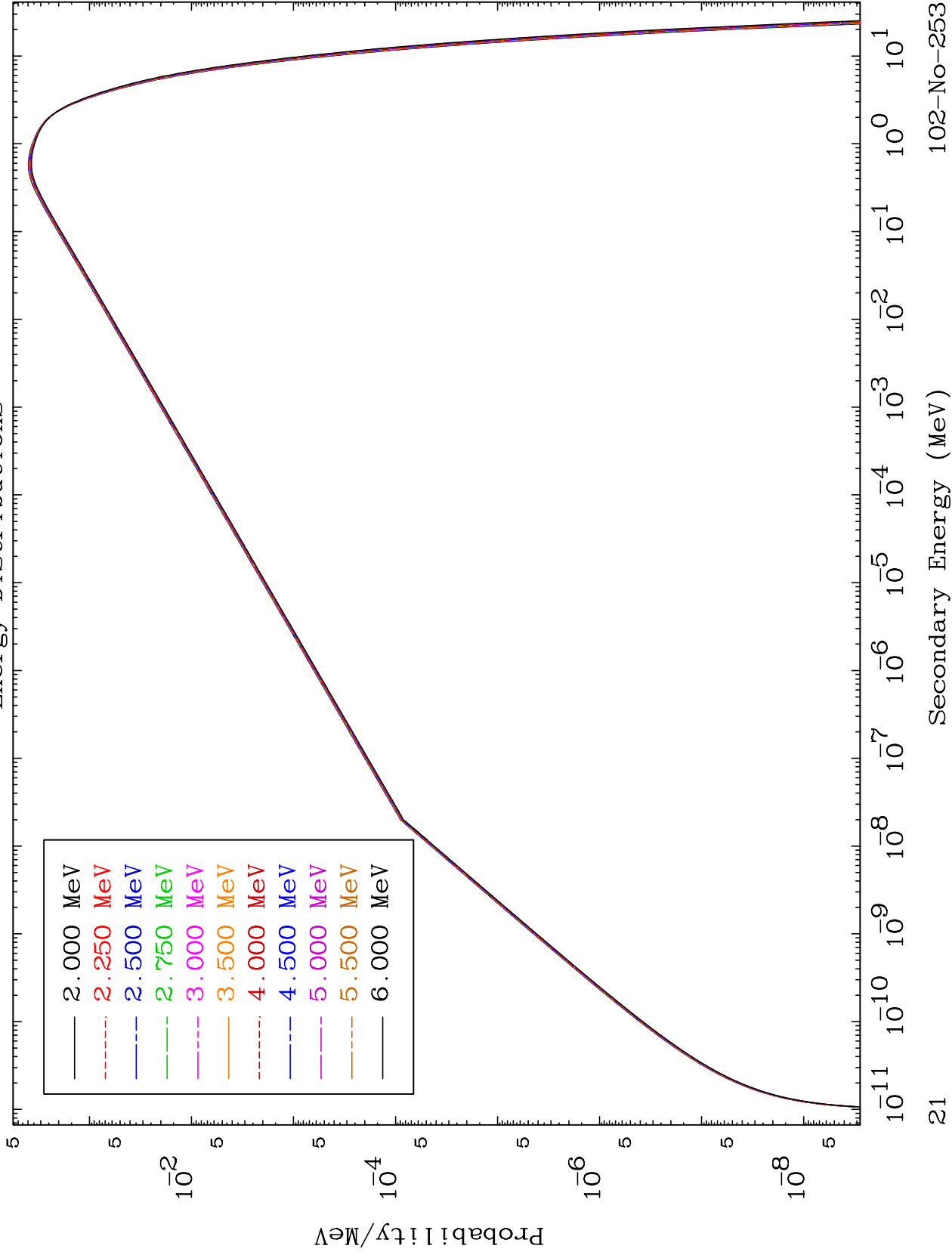
Incident Energy (MeV)

102-No-253

Fission Energy Distributions



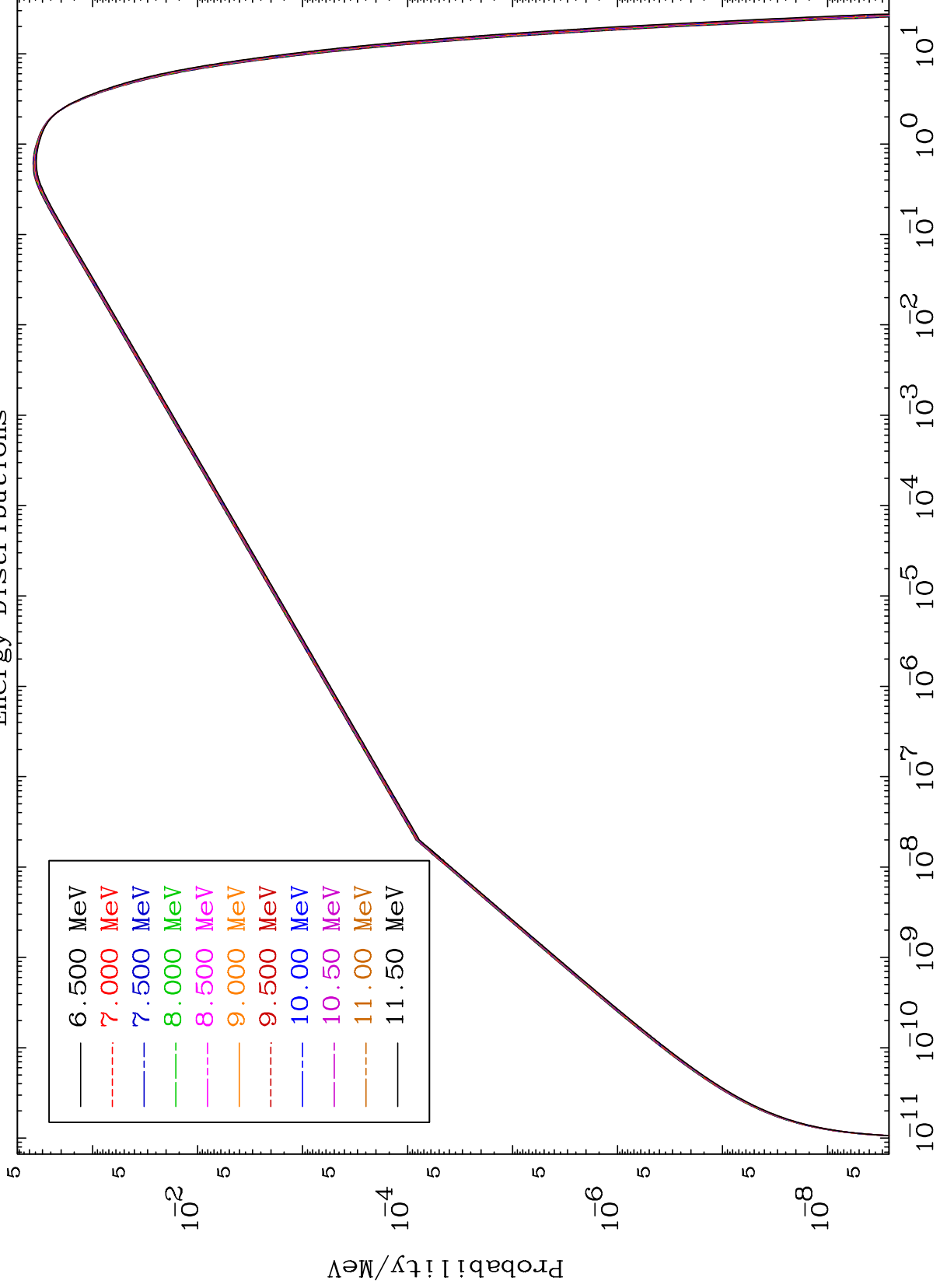
Fission Energy Distributions



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Fission Energy Distributions

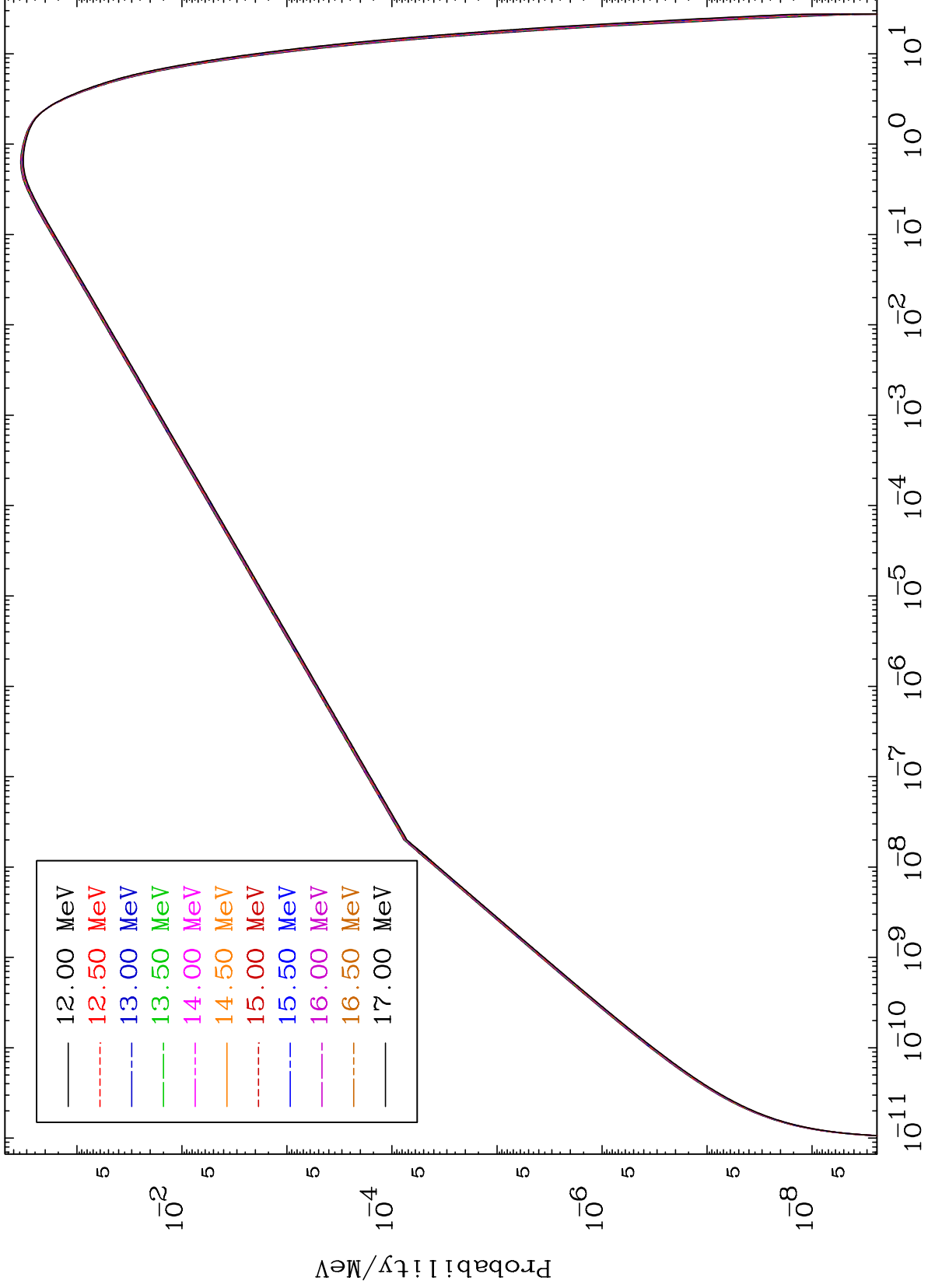
102-No-253



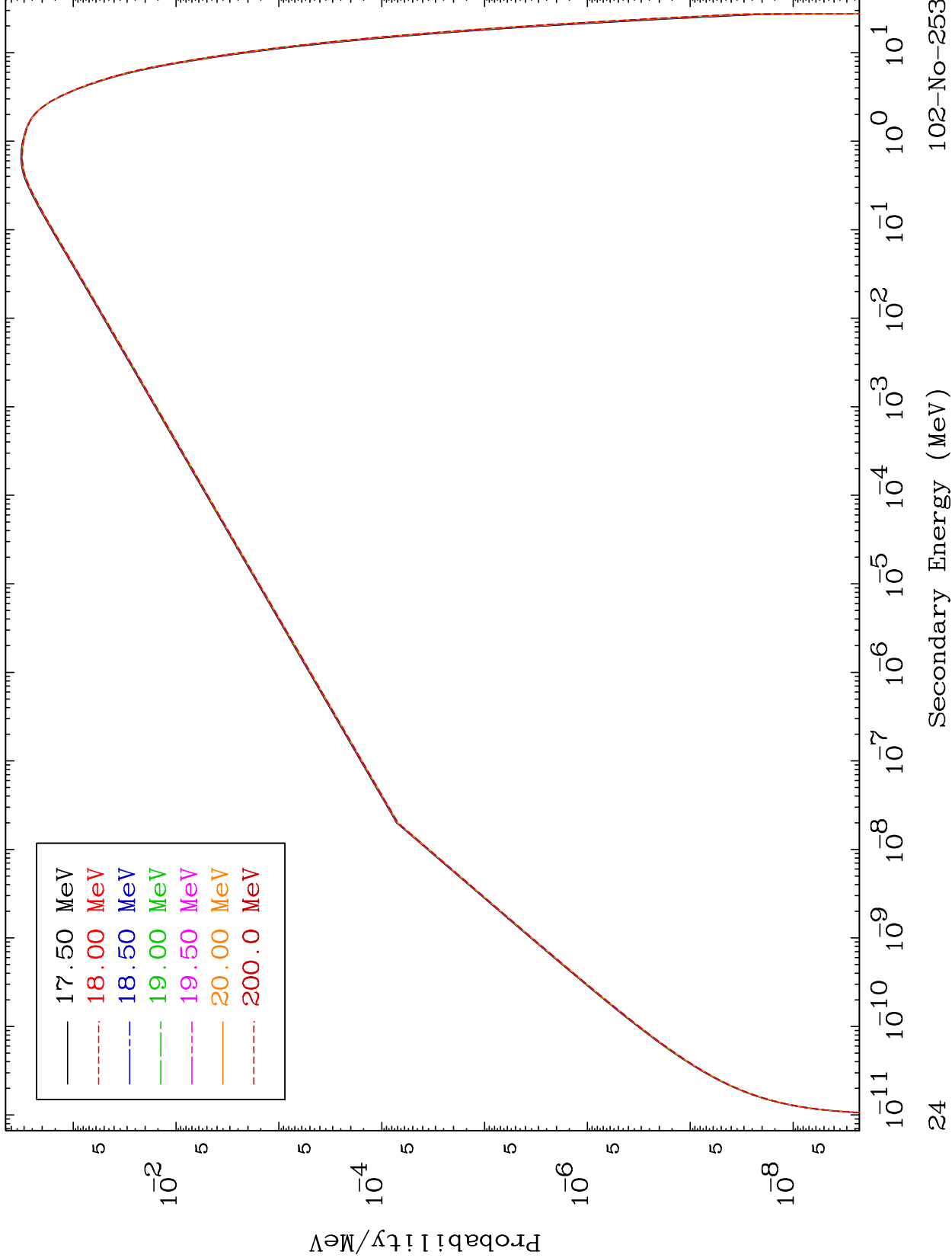
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Secondary Energy (MeV)

102-No-253



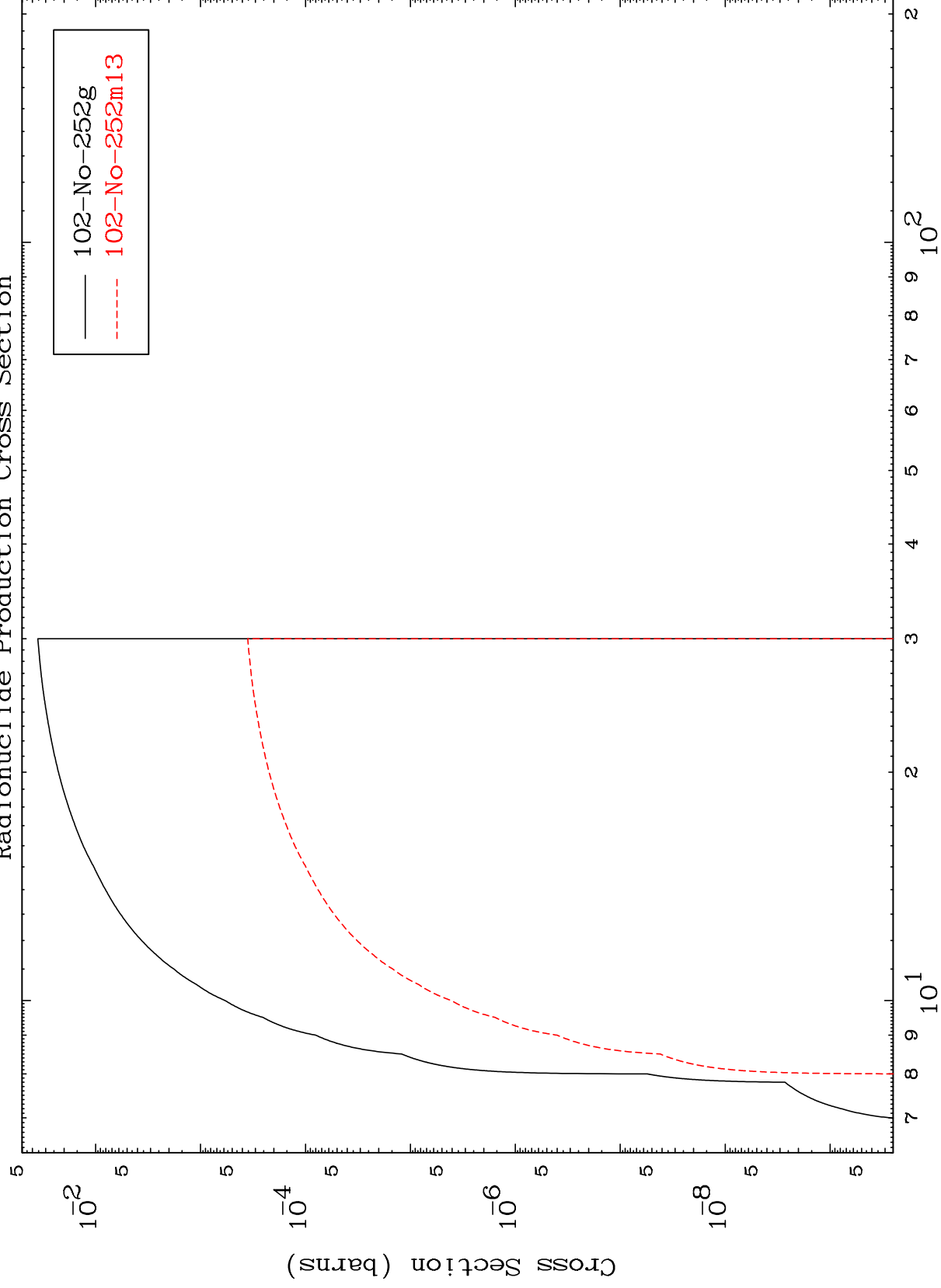
Fission Energy Distributions



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102-No-253

(n,2n)
Radionuclide Production Cross Section



102-No-253

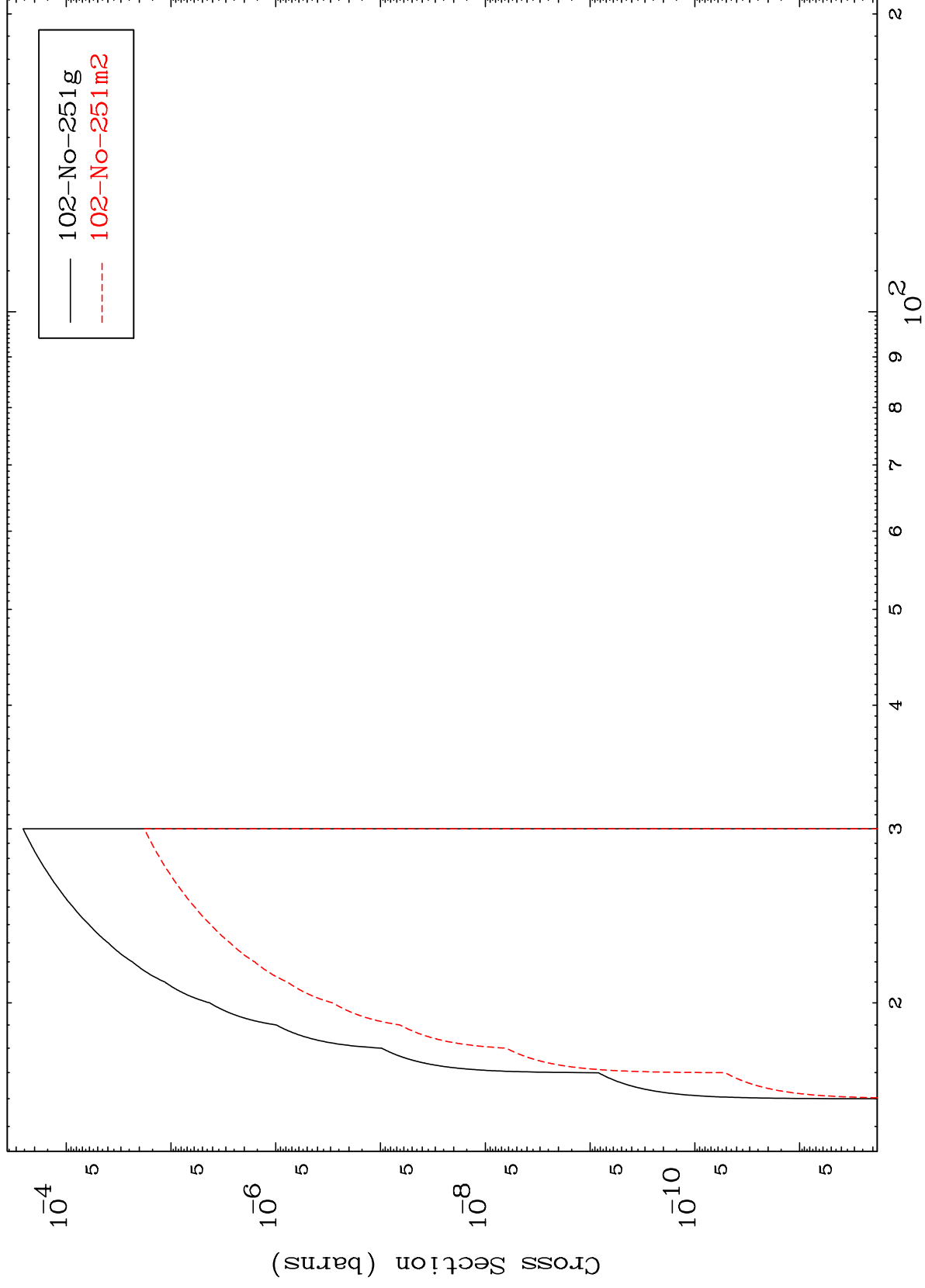
Incident Energy (MeV)

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(n,3n)
Radionuclide Production Cross Section



26

Incident Energy (MeV)

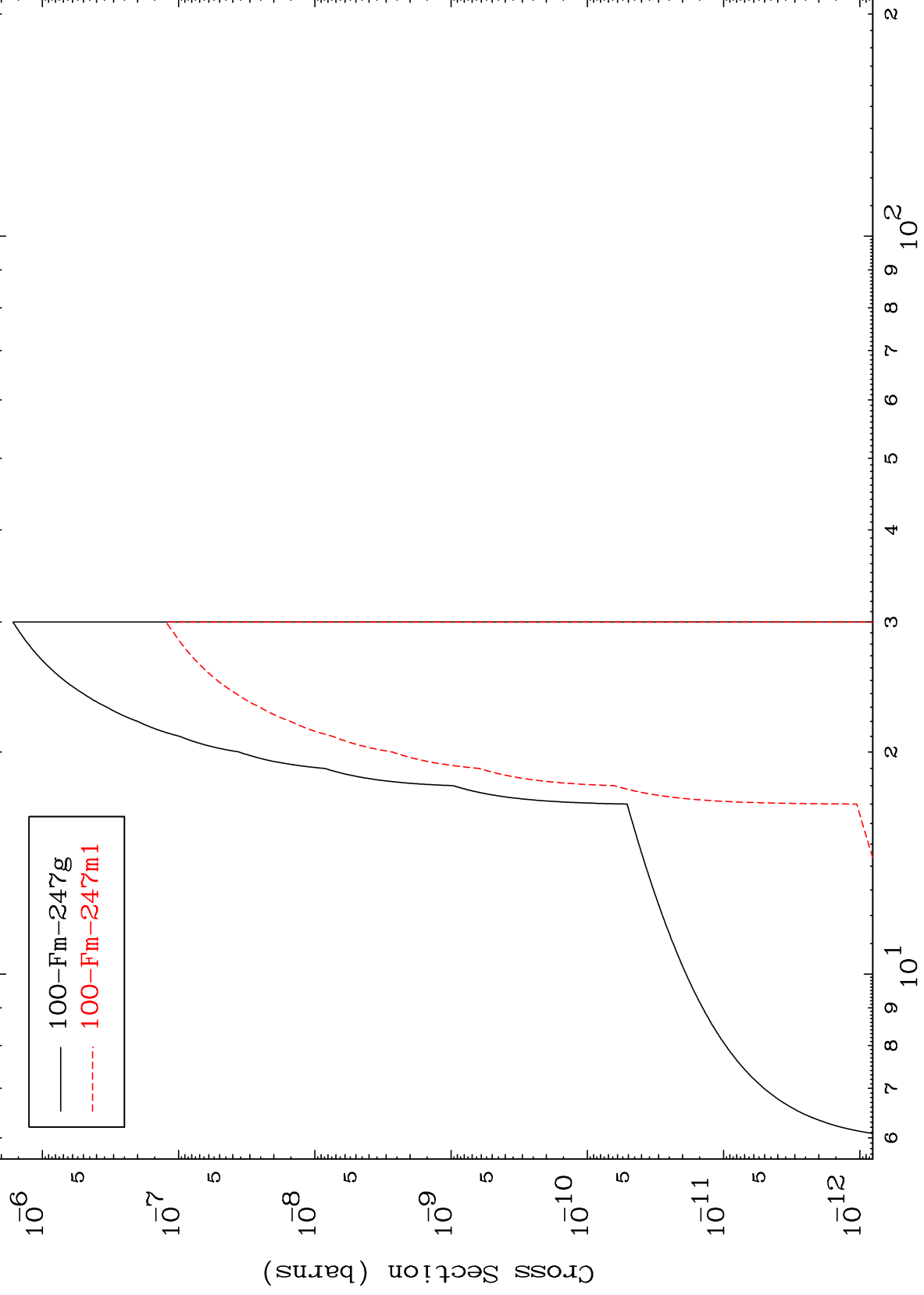
102-No-253

MAT 9971

(n,3n) α

102-No-253

Radionuclide Production Cross Section

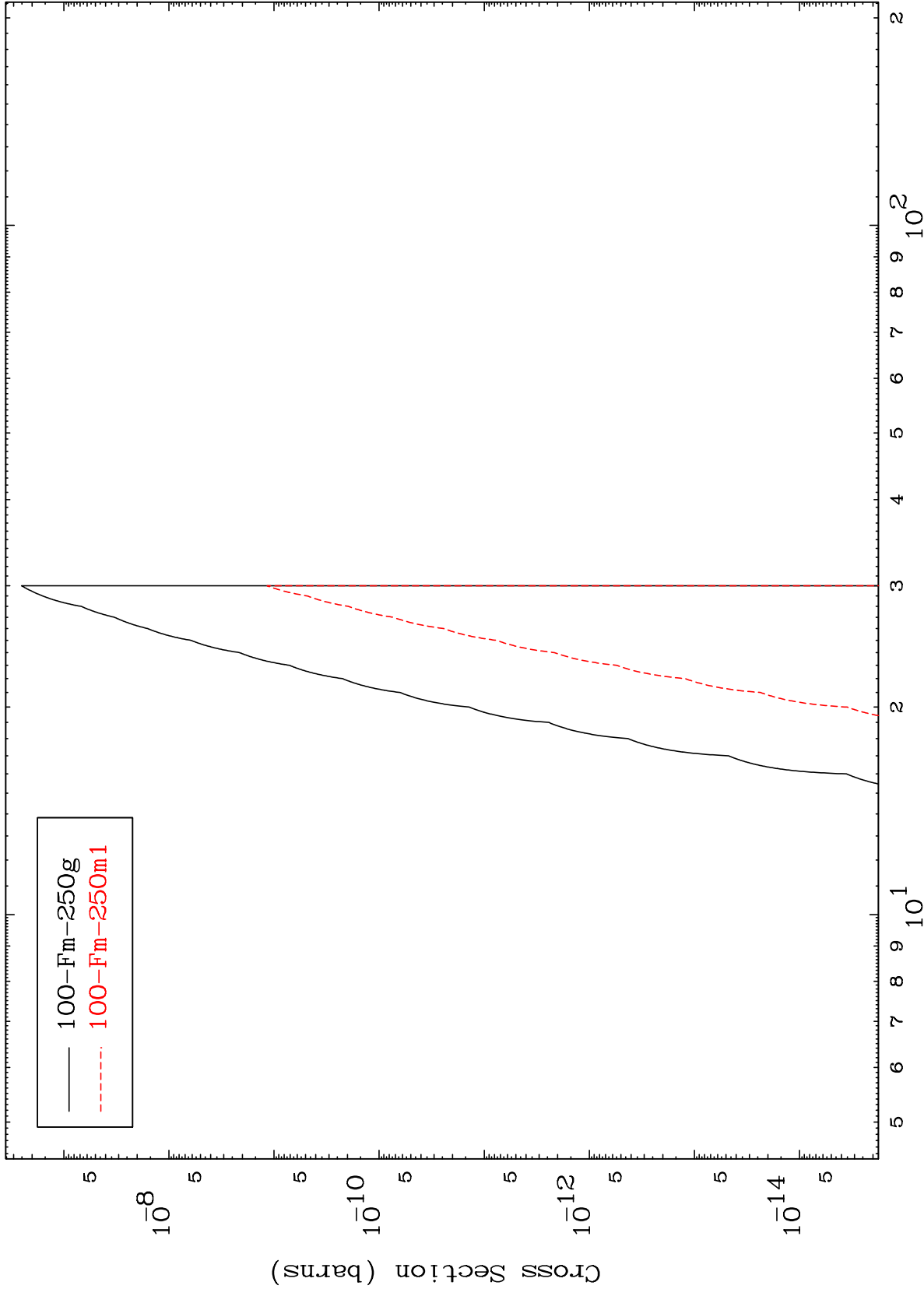


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Incident Energy (MeV)

102-No-253

Radionuclide Production Cross Section

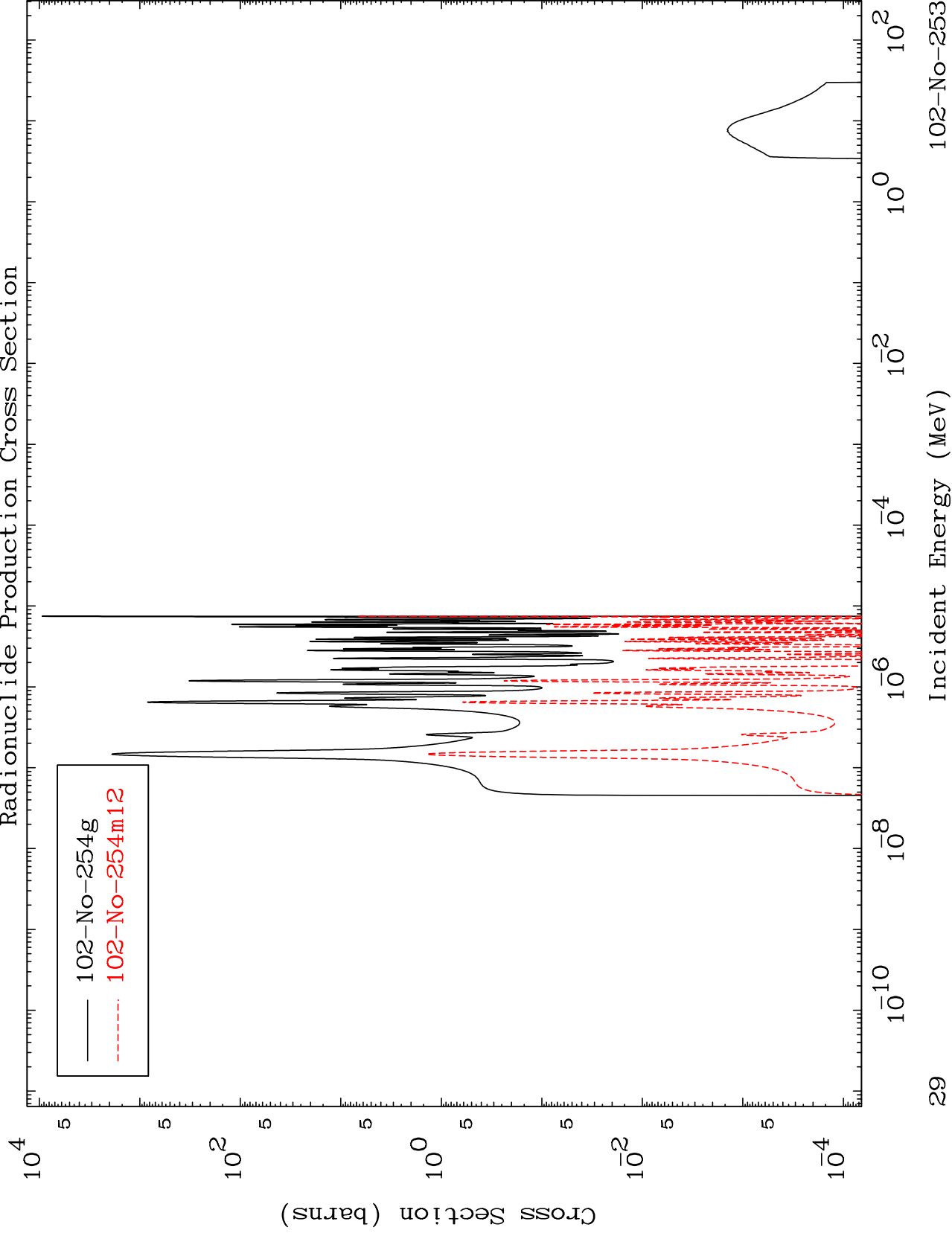


— 100-Fm-250g
- - - 100-Fm-250m1

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102-No-253

Radionuclide Production Cross Section
(n, γ)



29

102-No-253

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102-No-253

(n, α)
Radionuclide Production Cross Section

