

Program EVALPLOT
(Version 2021-1)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

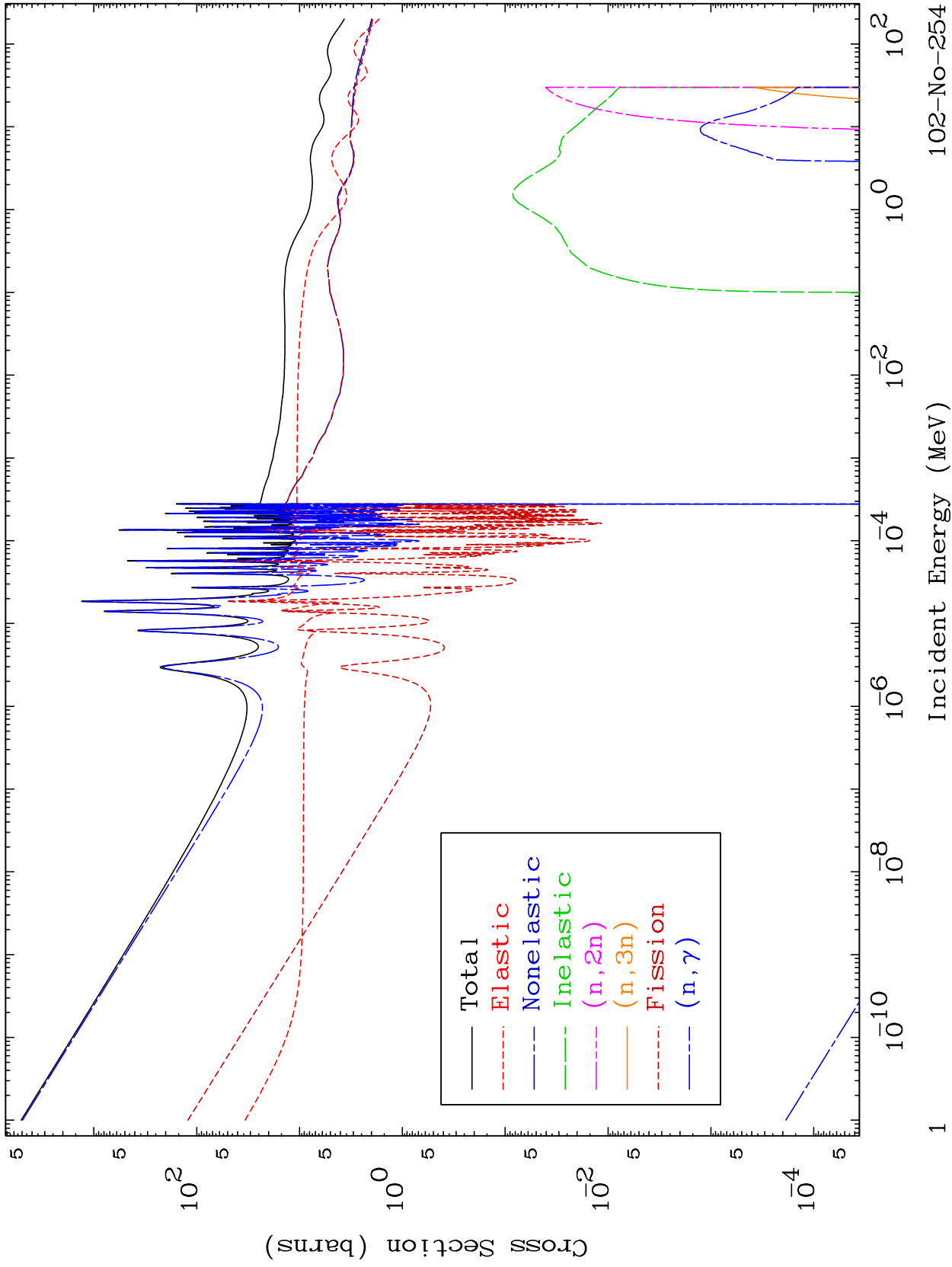
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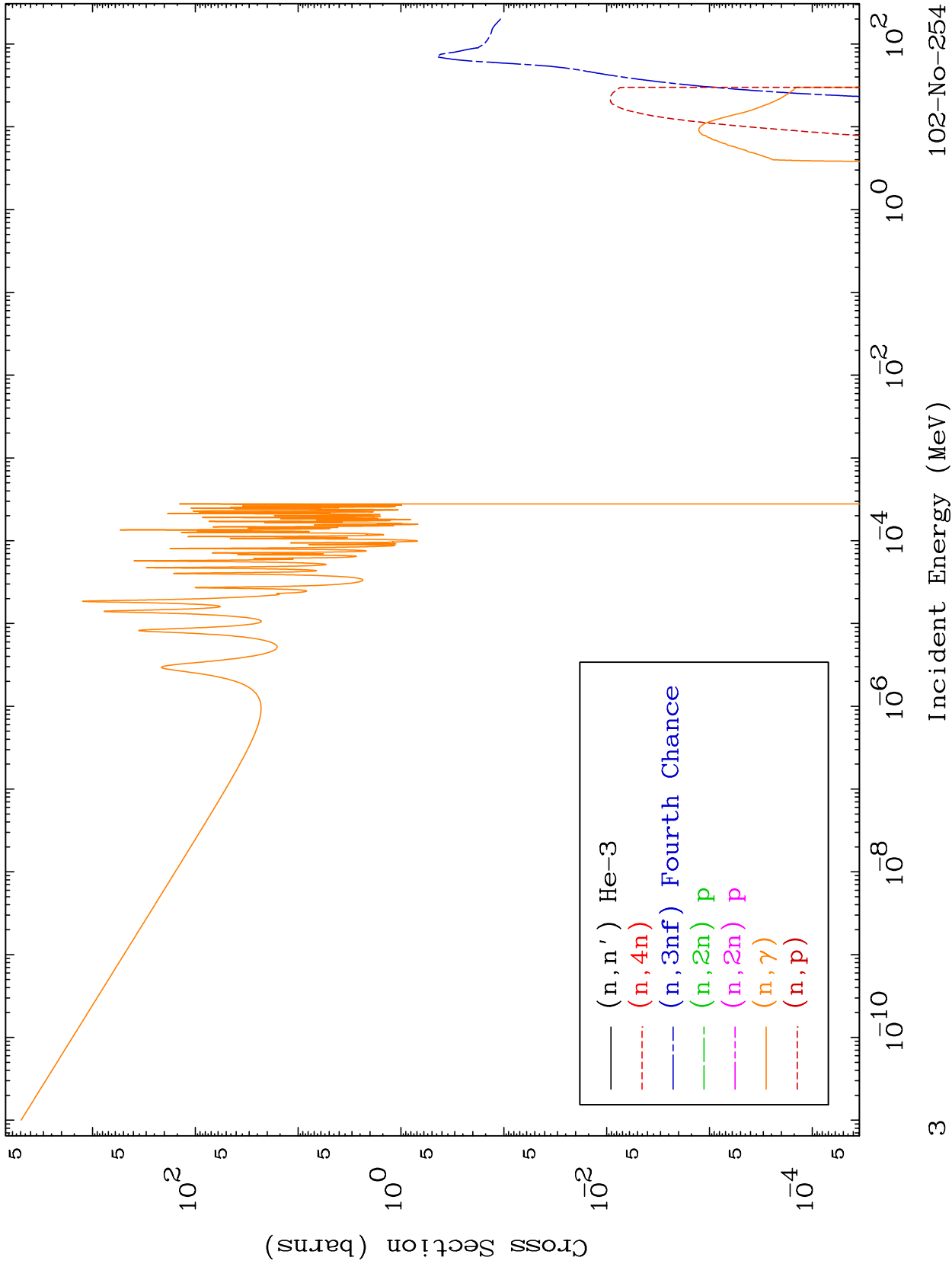
Press Mouse Button to Start

MAT 9972

Neutron Major
293 Kelvin Cross Sections

102-No-254

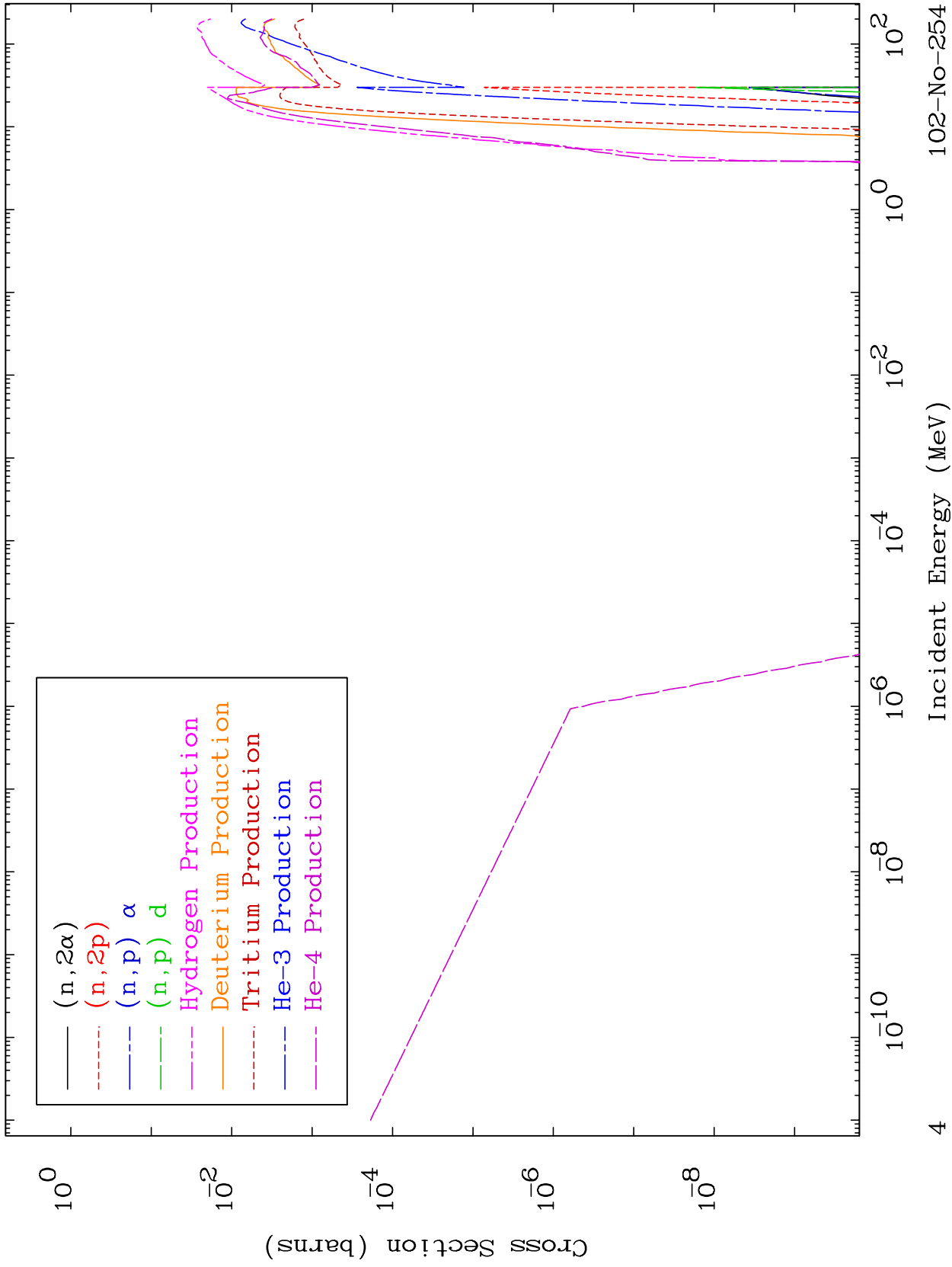




MAT 9972

Neutron Absorption
293 Kelvin Cross Sections

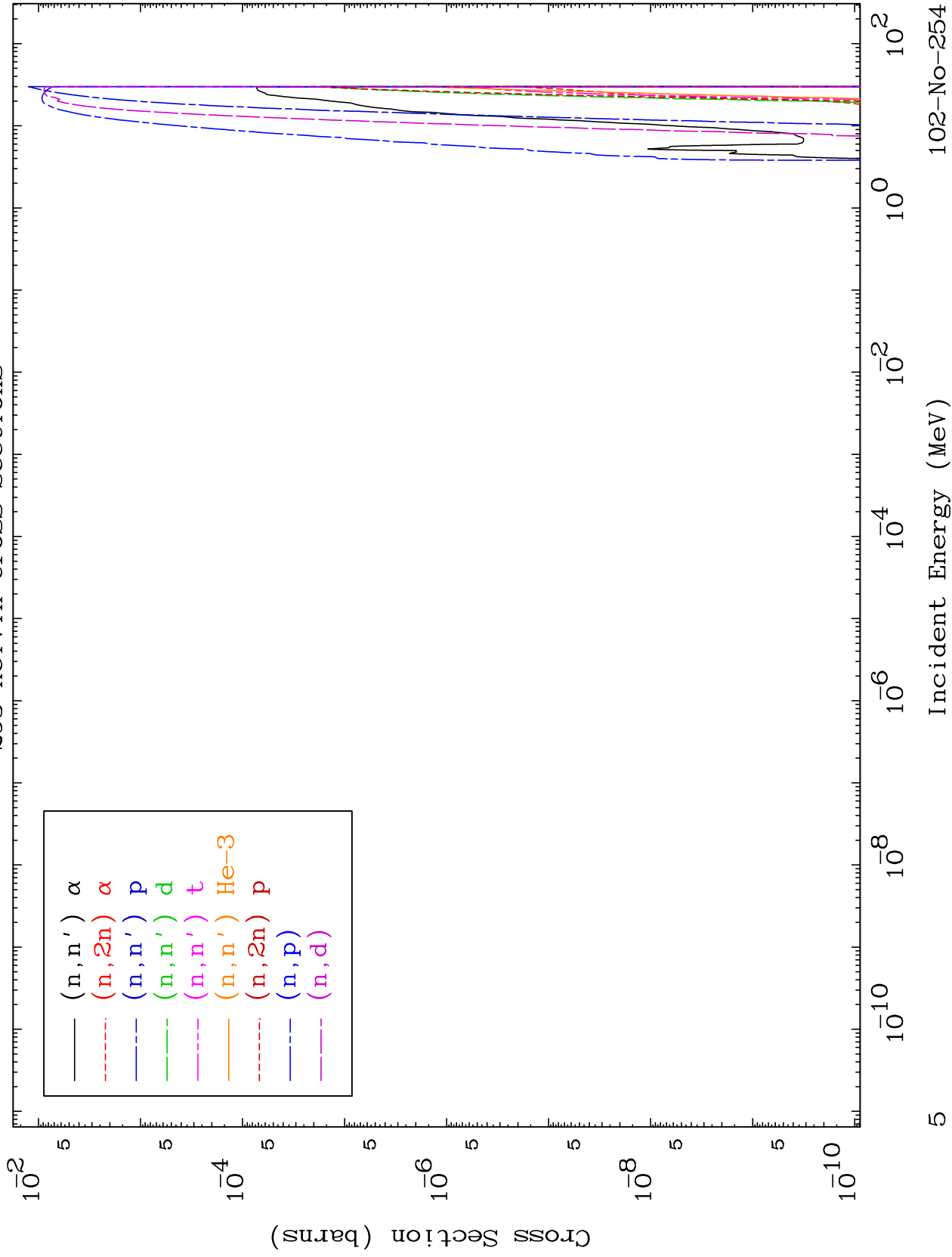
102-No-254



MAT 9972

Charged Particle
293 Kelvin Cross Sections

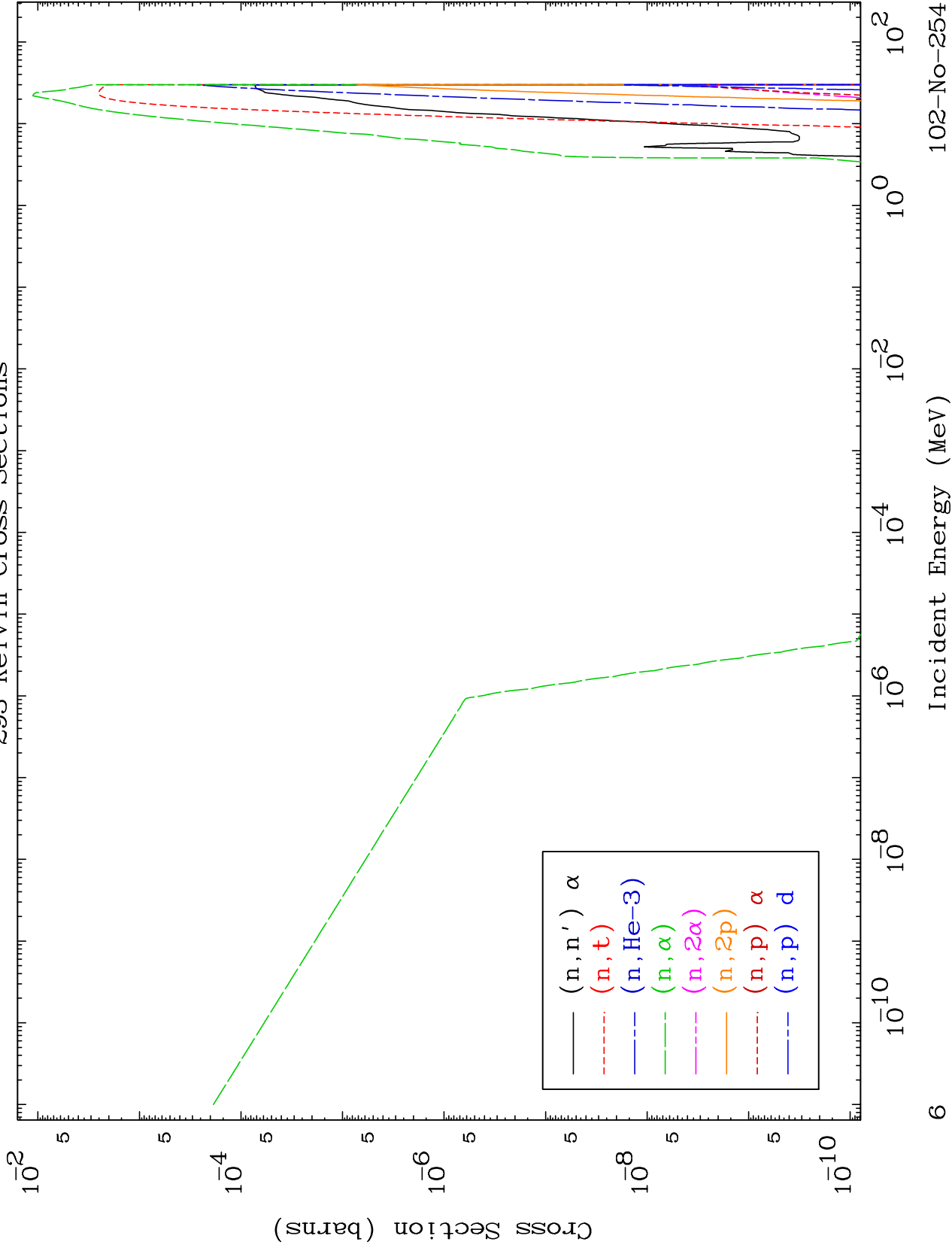
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MAT 9972

Charged Particle
293 Kelvin Cross Sections

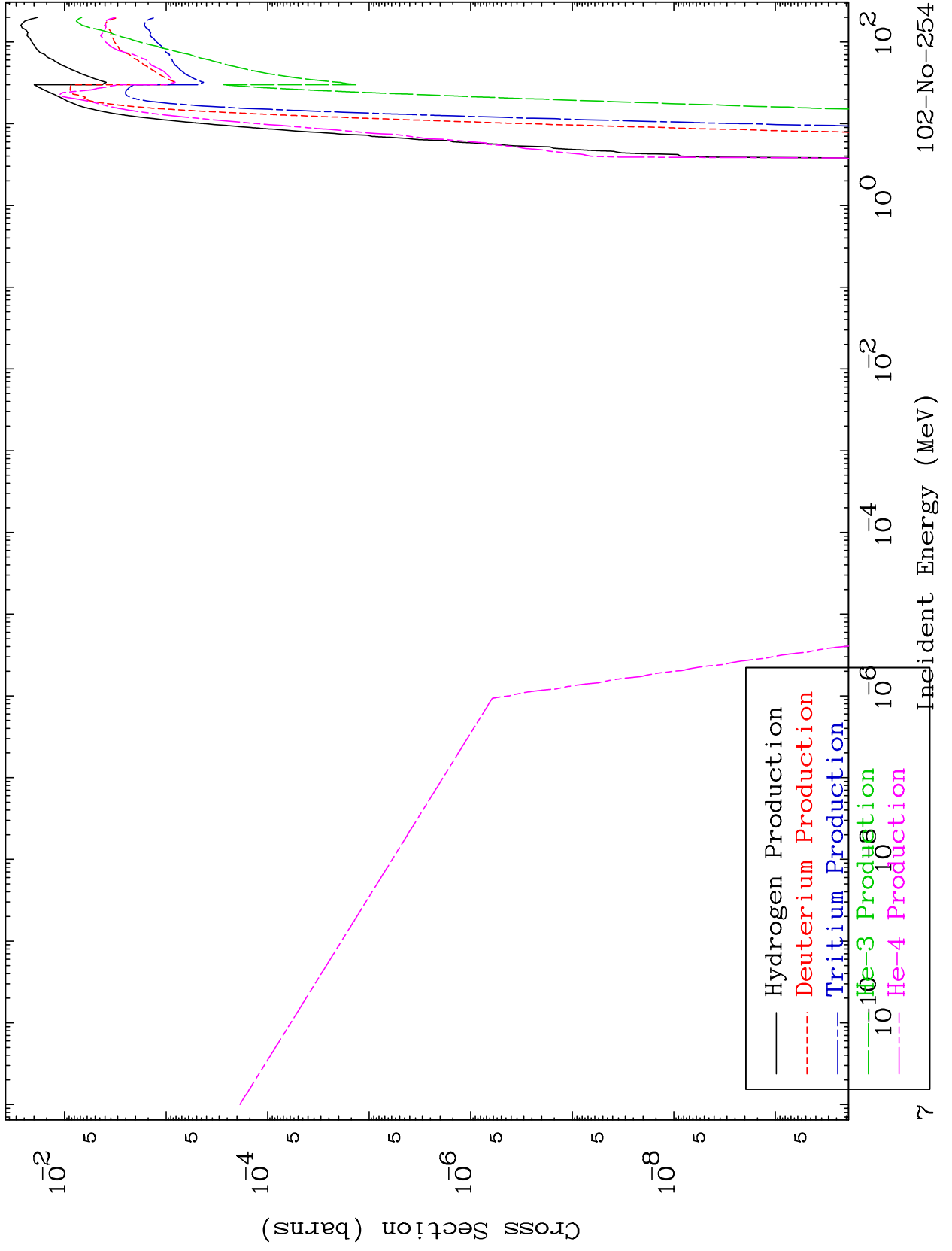
102-No-254



MAT 9972

Particle Production
293 Kelvin Cross Sections

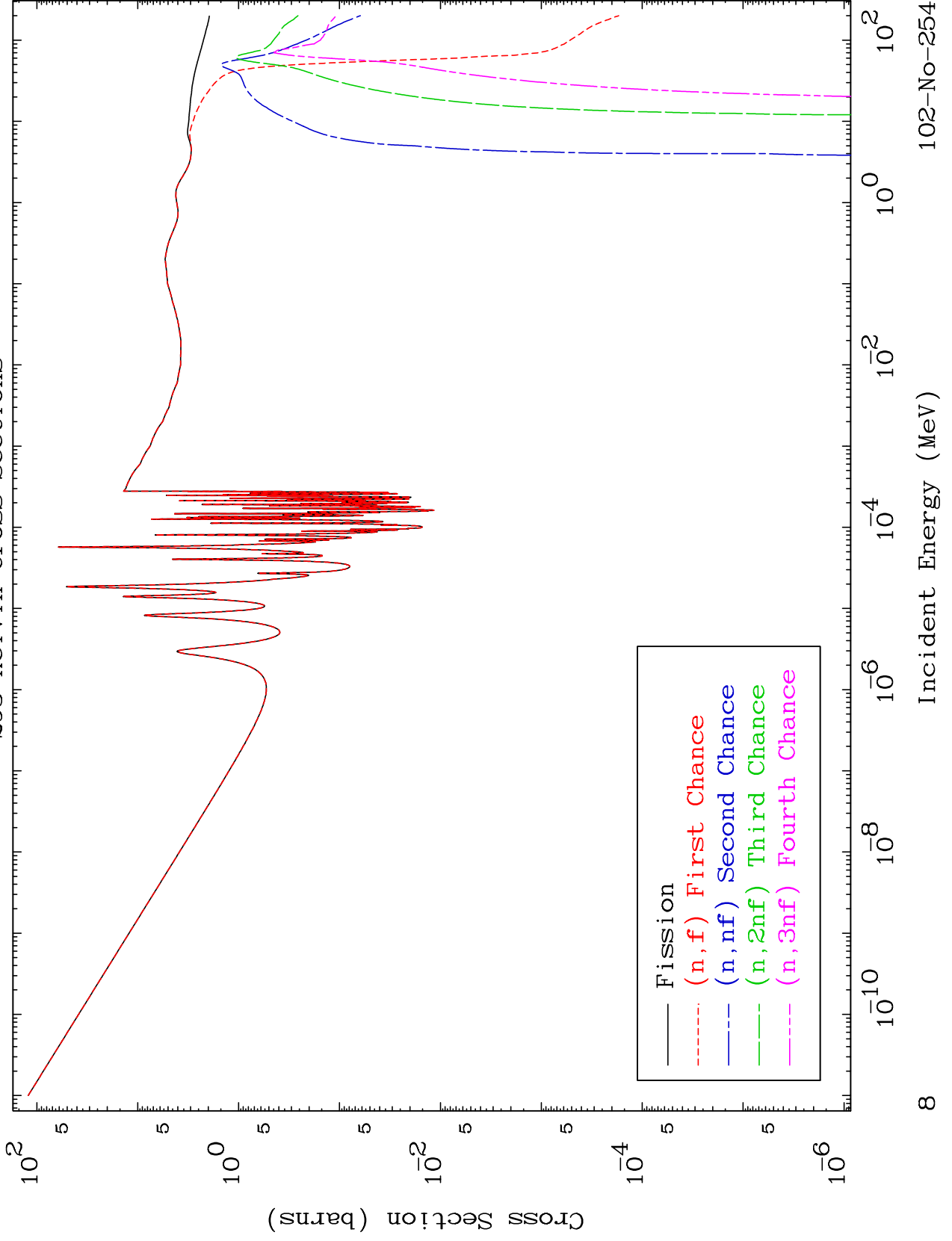
102-No-254



MAT 9972

Fission
293 Kelvin Cross Sections

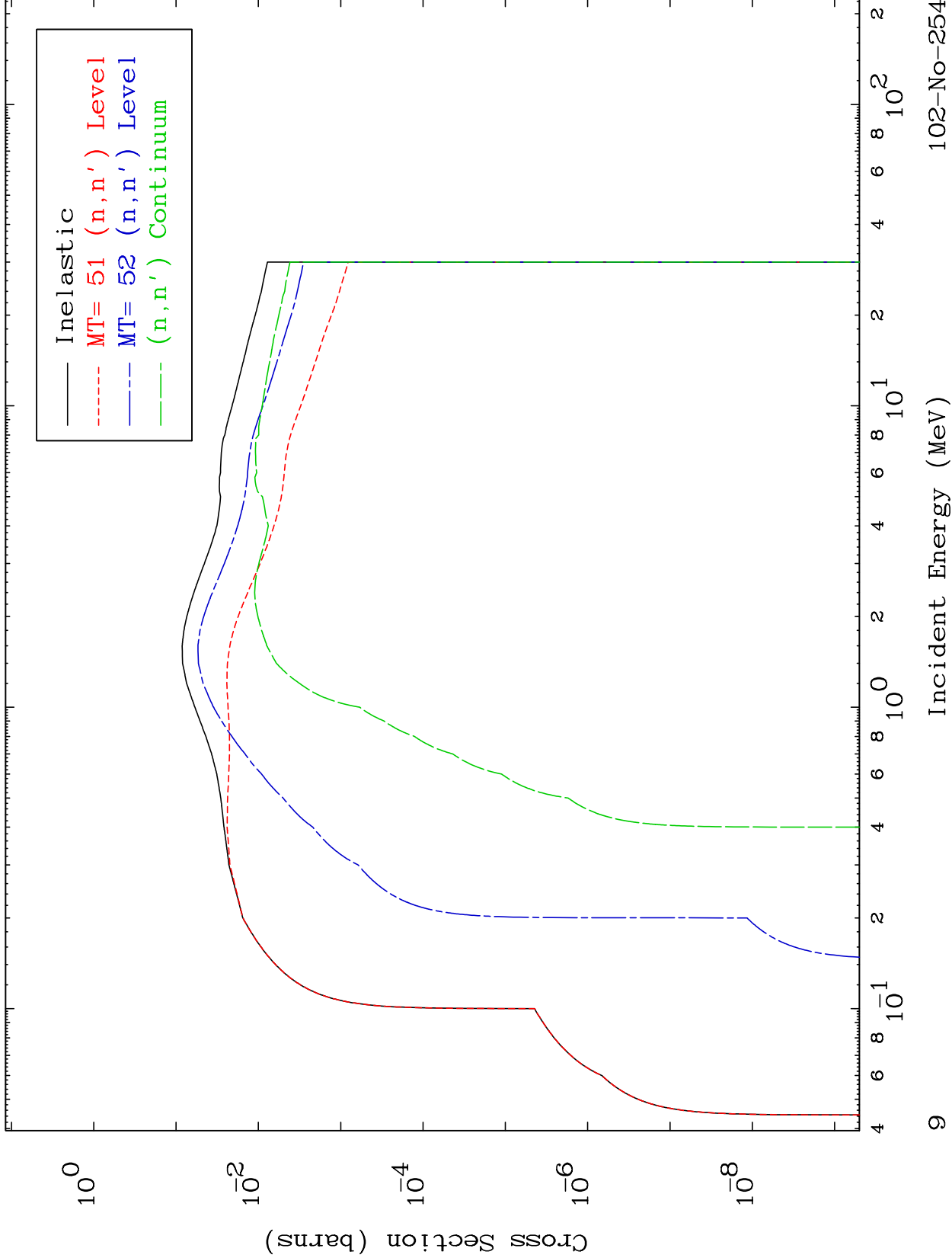
102-No-254



MAT 9972

(n,n') Levels
293 Kelvin Cross Sections

102-No-254



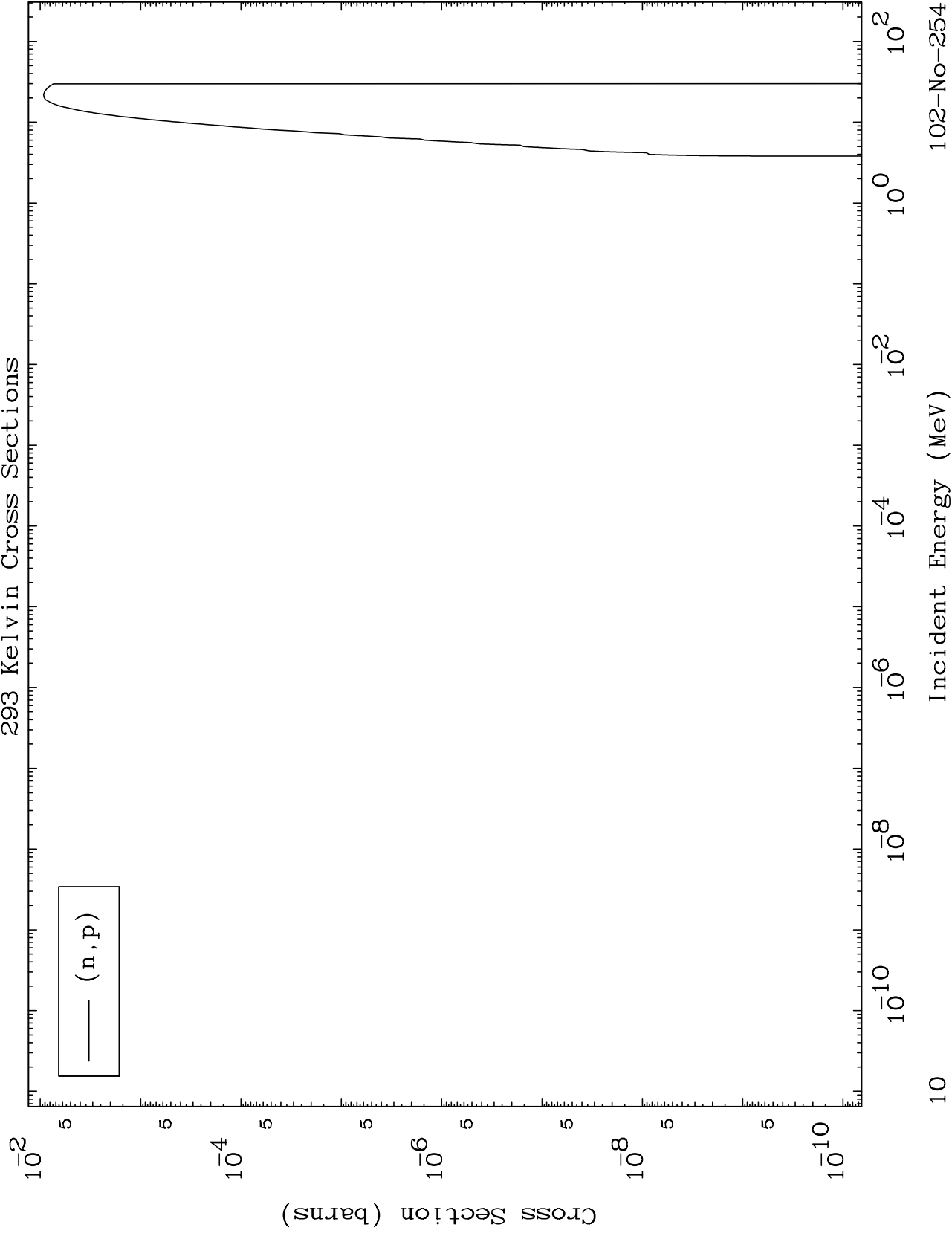
9

102-No-254

MAT 9972

(n,p) Levels
293 Kelvin Cross Sections

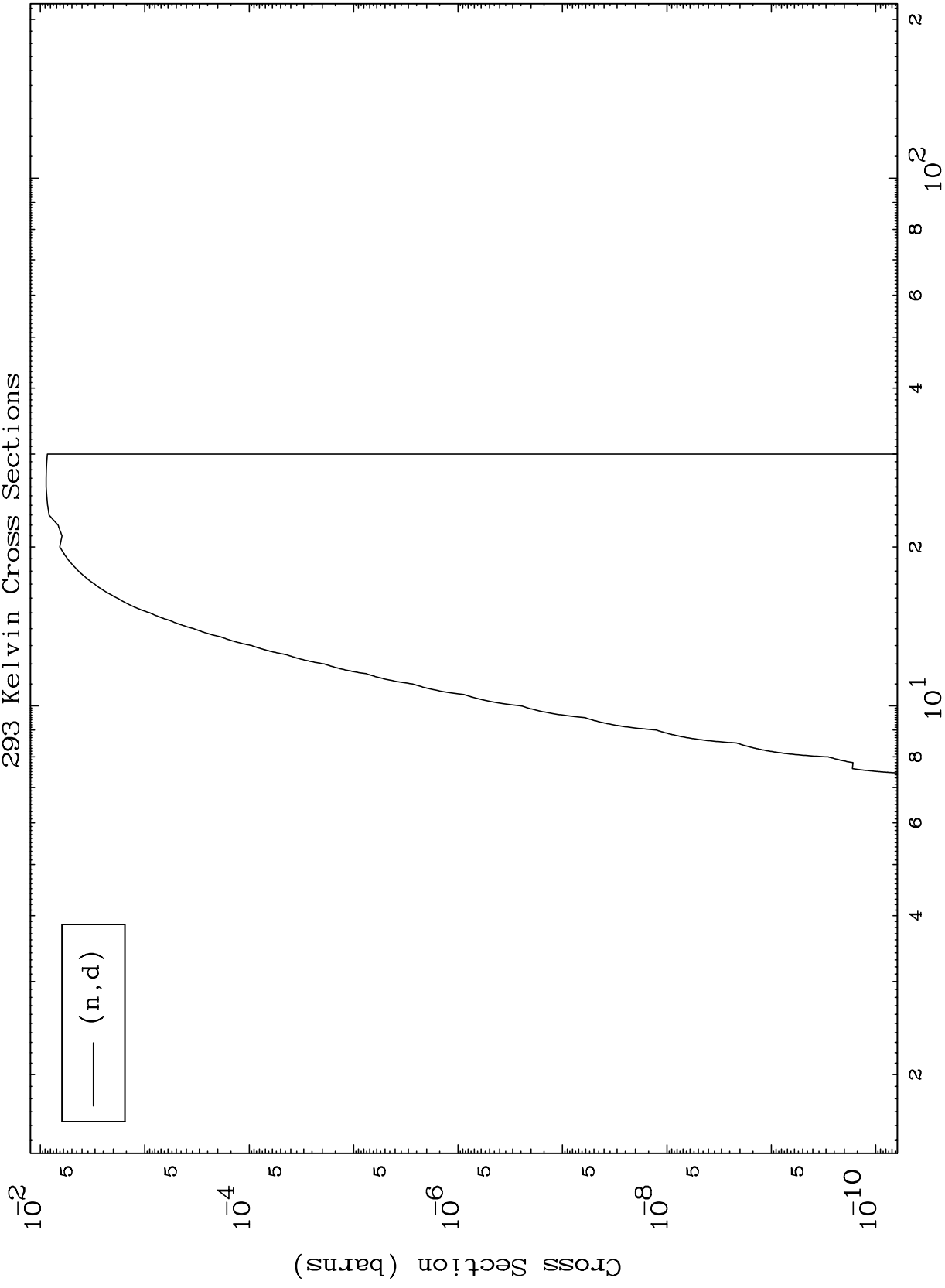
102-No-254



MAT 9972

(n,d) Levels
293 Kelvin Cross Sections

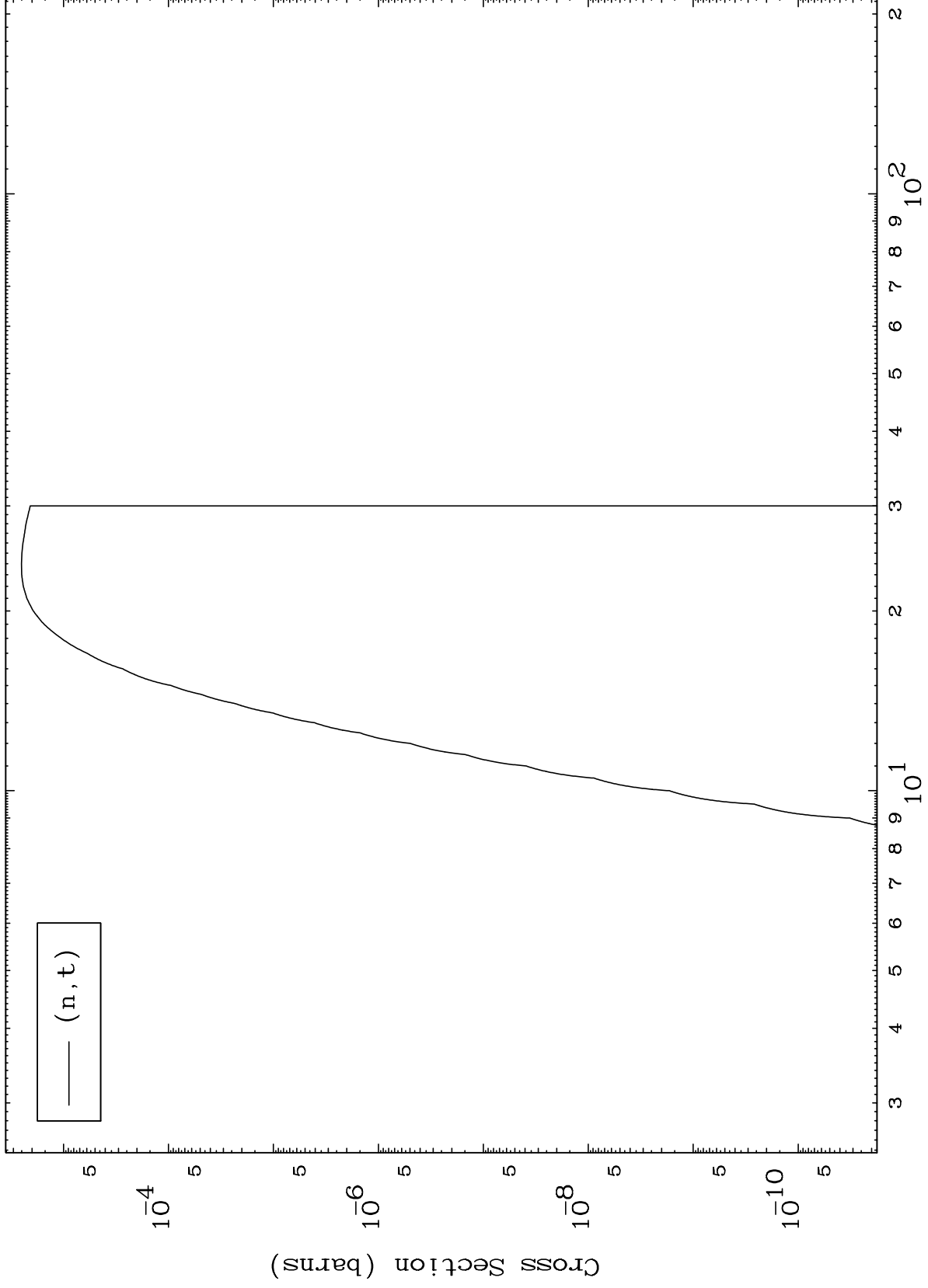
102-No-254



MAT 9972

(n,t) Levels
293 Kelvin Cross Sections

102-No-254



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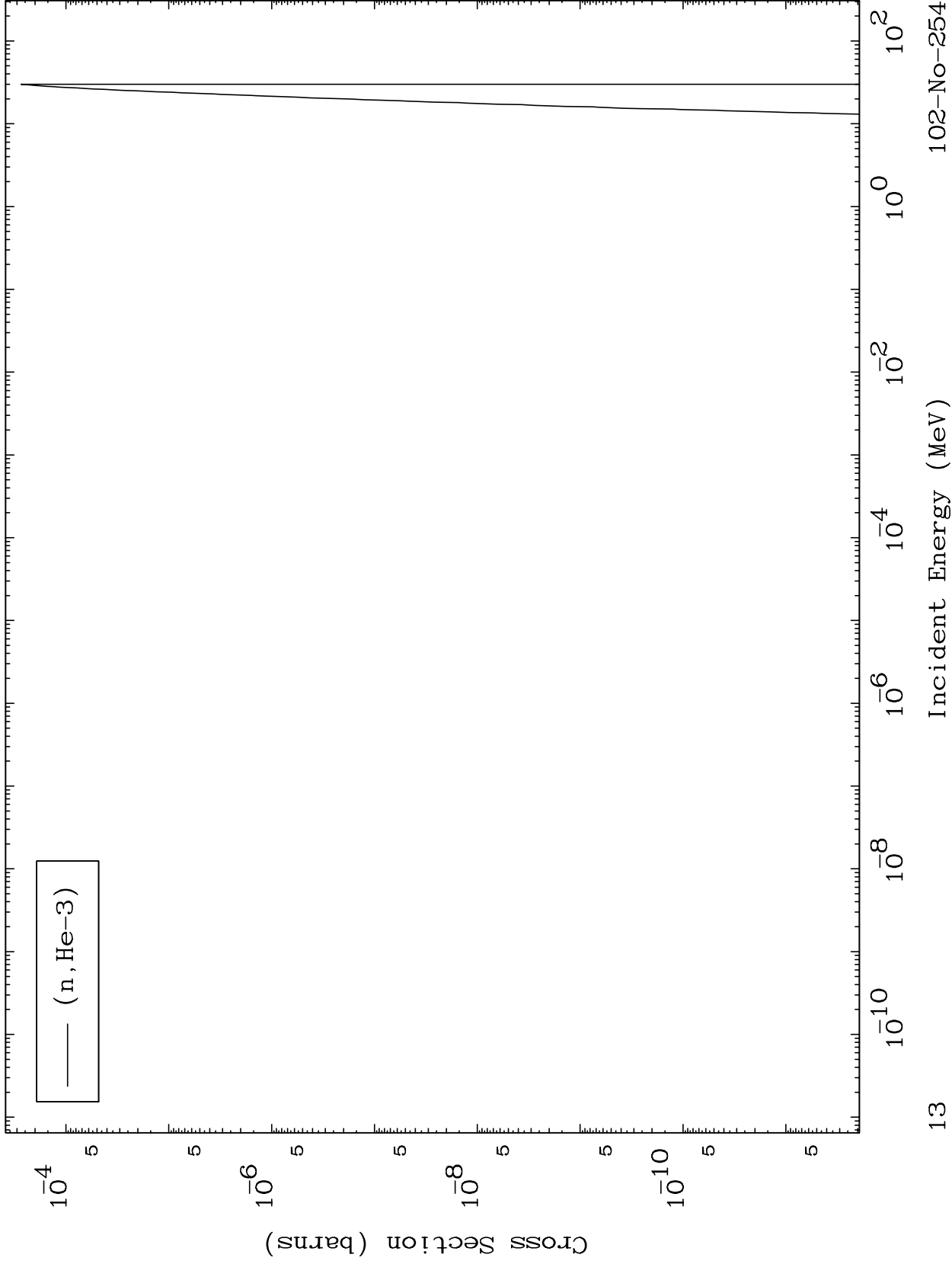
Incident Energy (MeV)

102-No-254

MAT 9972

(n,He3) Levels
293 Kelvin Cross Sections

102-No-254

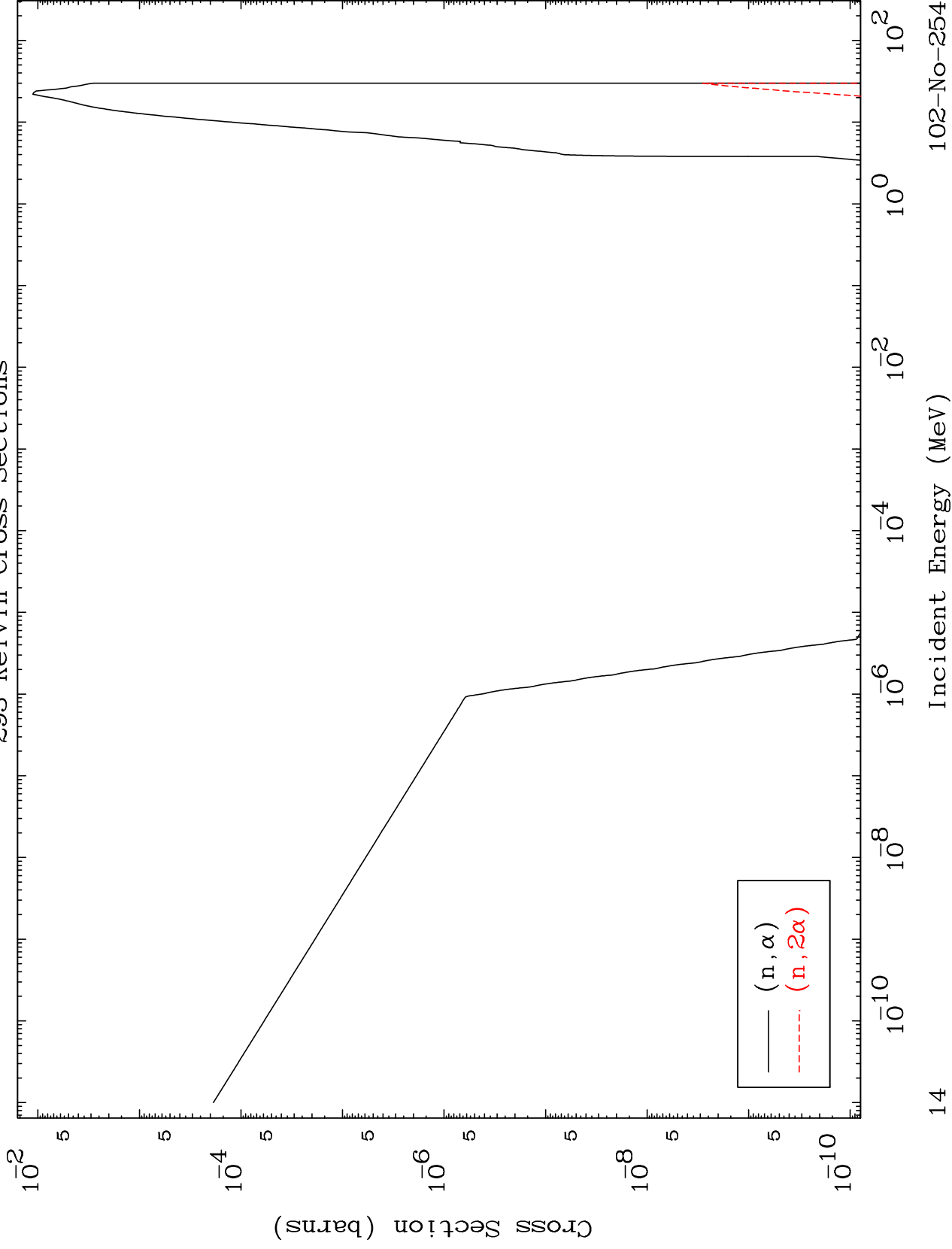


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(n, α) Levels
293 Kelvin Cross Sections

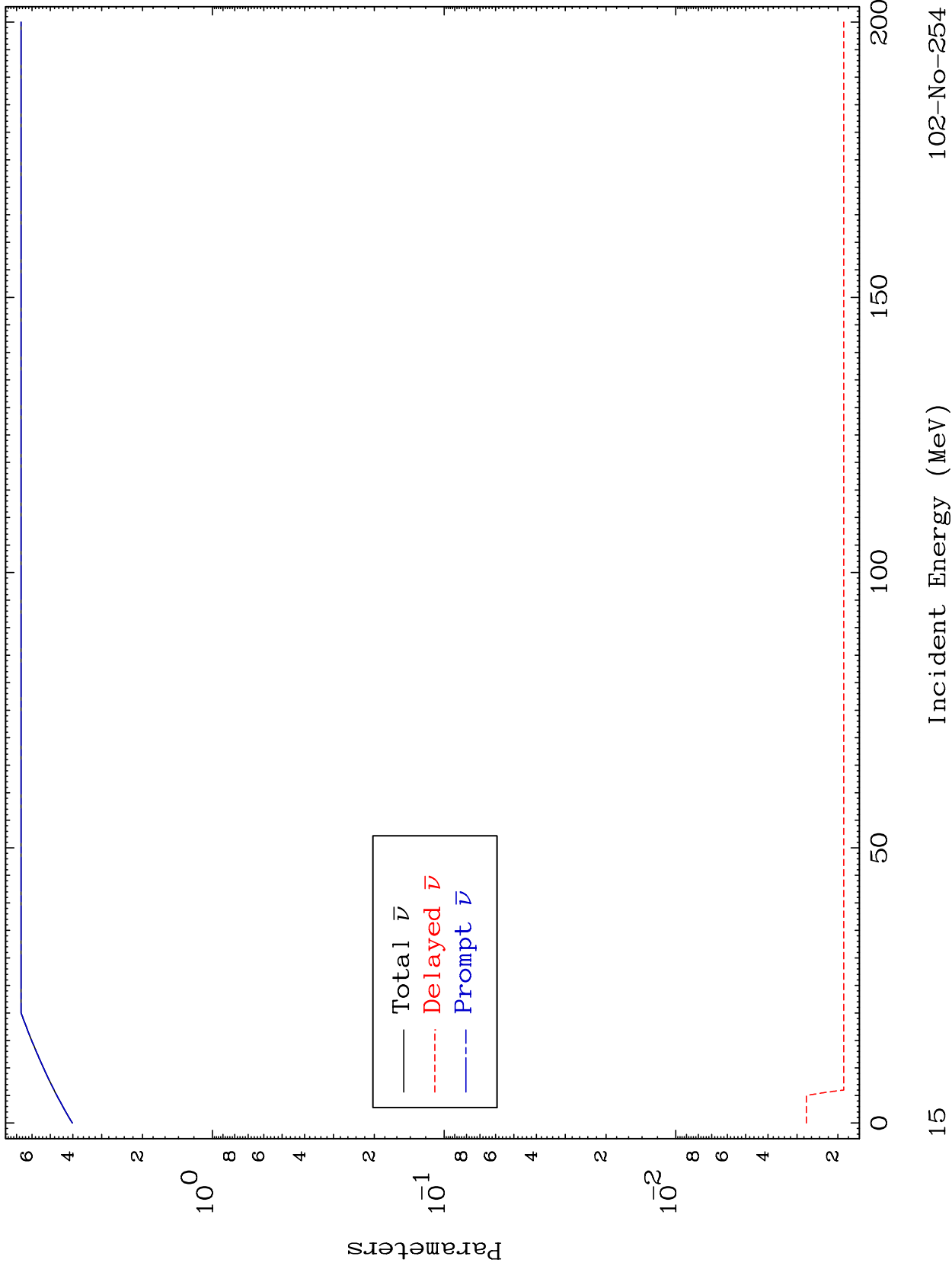
102-No-254



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Energy Release
Parameters

102-No-254



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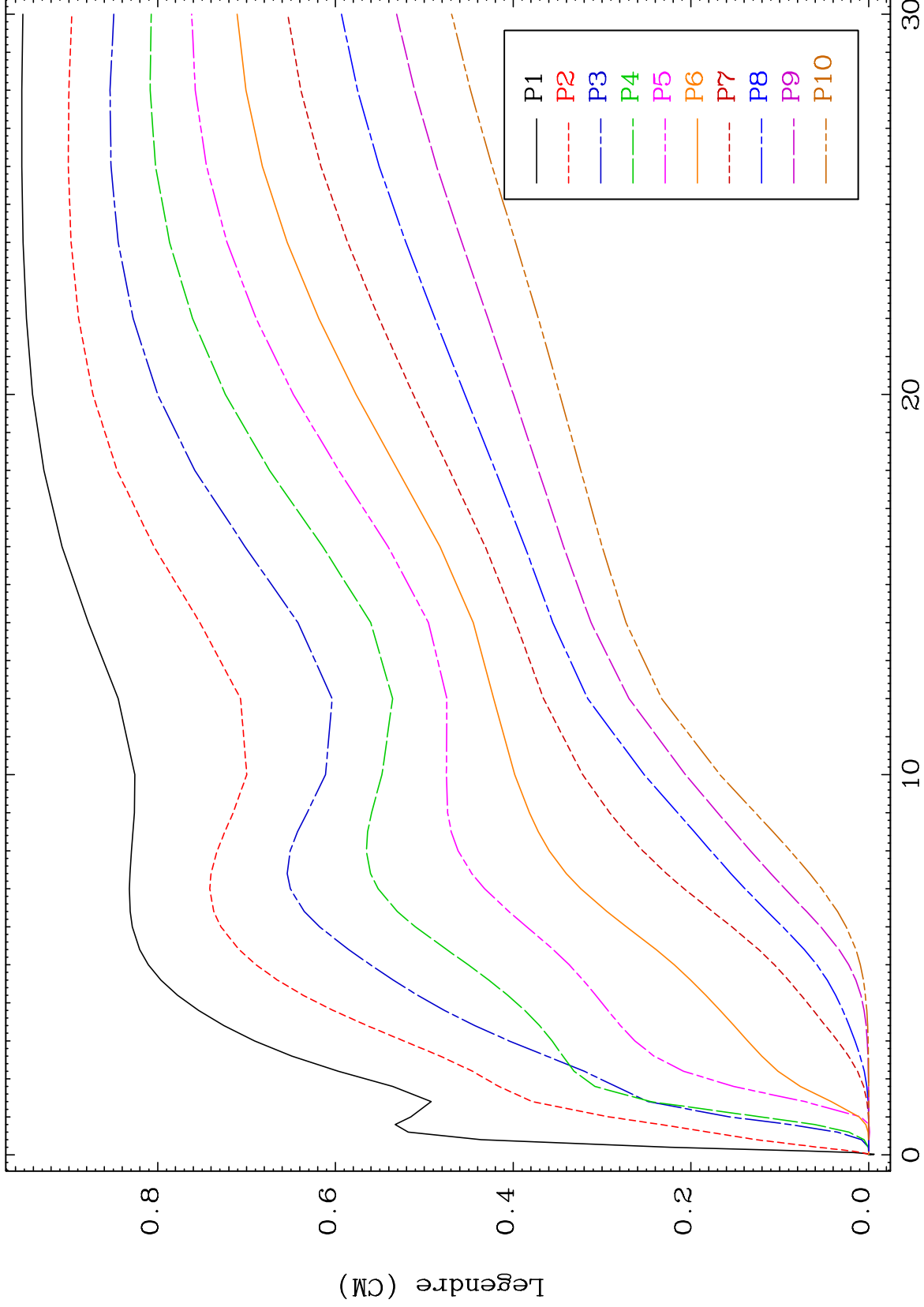
Incident Energy (MeV)

102-No-254

MAT 9972

Elastic Legendre Coefficients

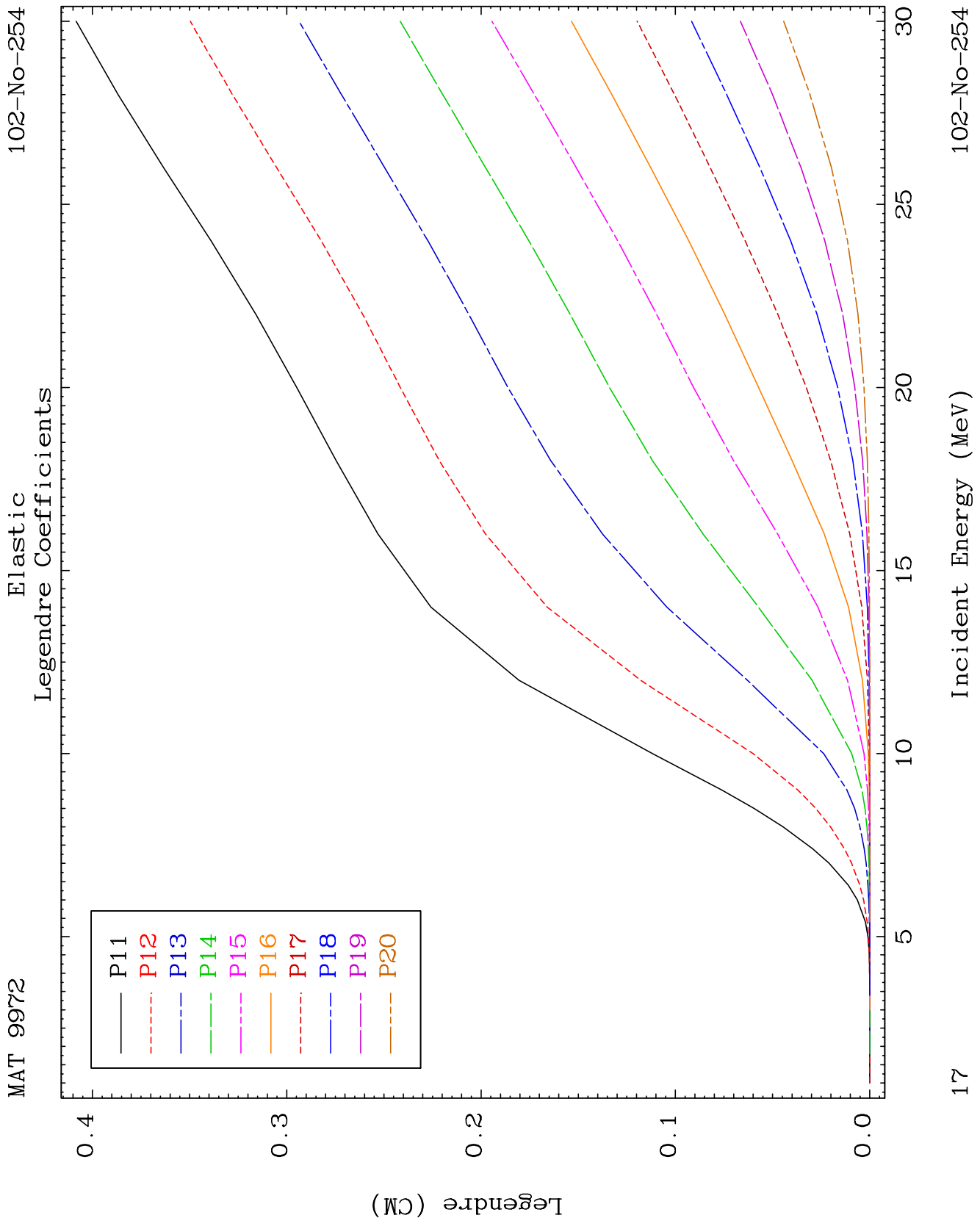
102-No-254



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Incident Energy (MeV)

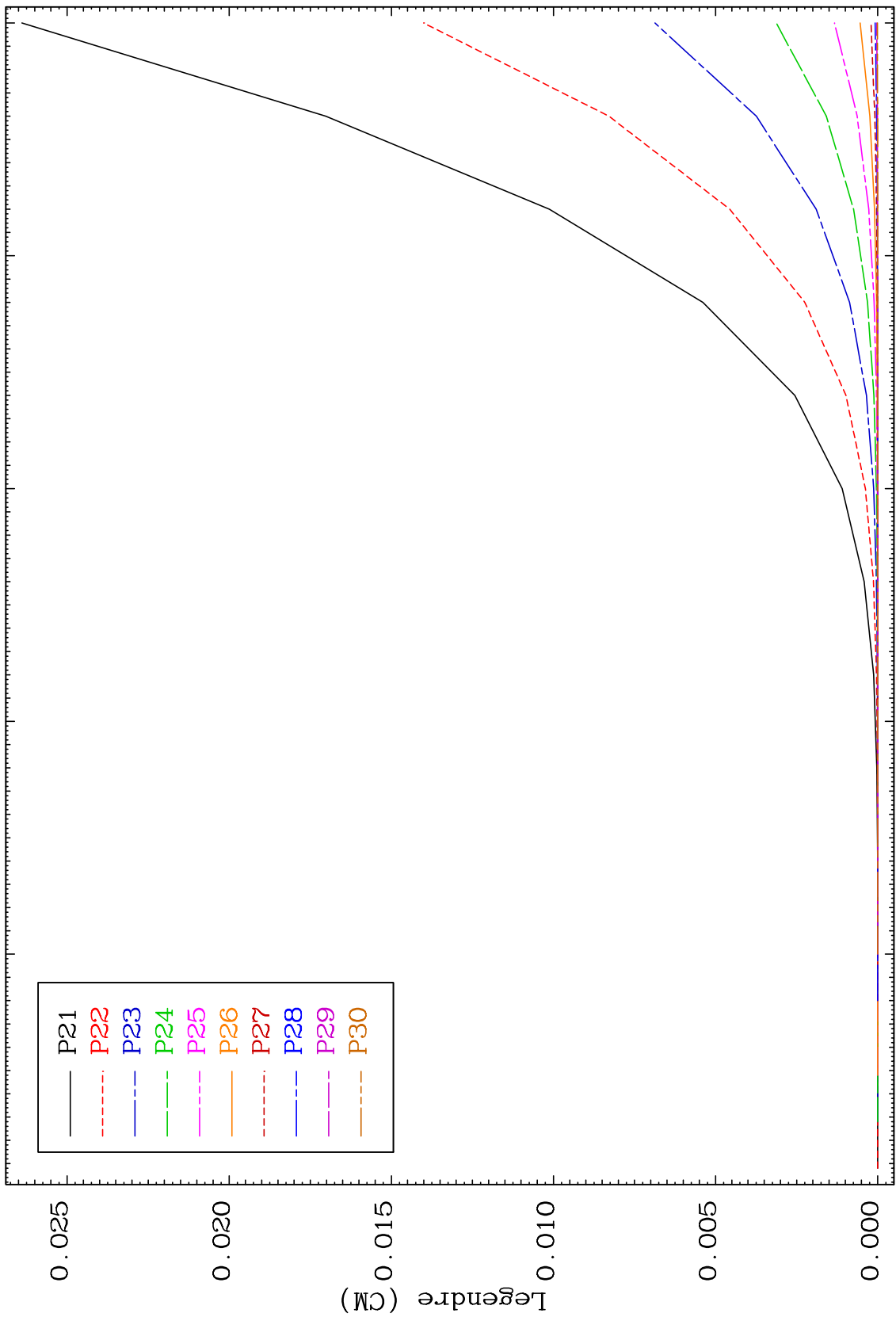
102-No-254



MAT 9972

Elastic Legendre Coefficients

102-No-254



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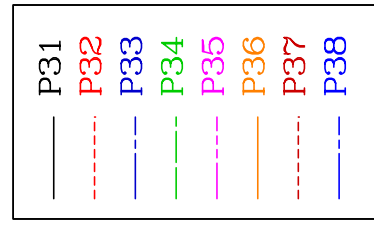
Incident Energy (MeV)

102-No-254

MAT 9972

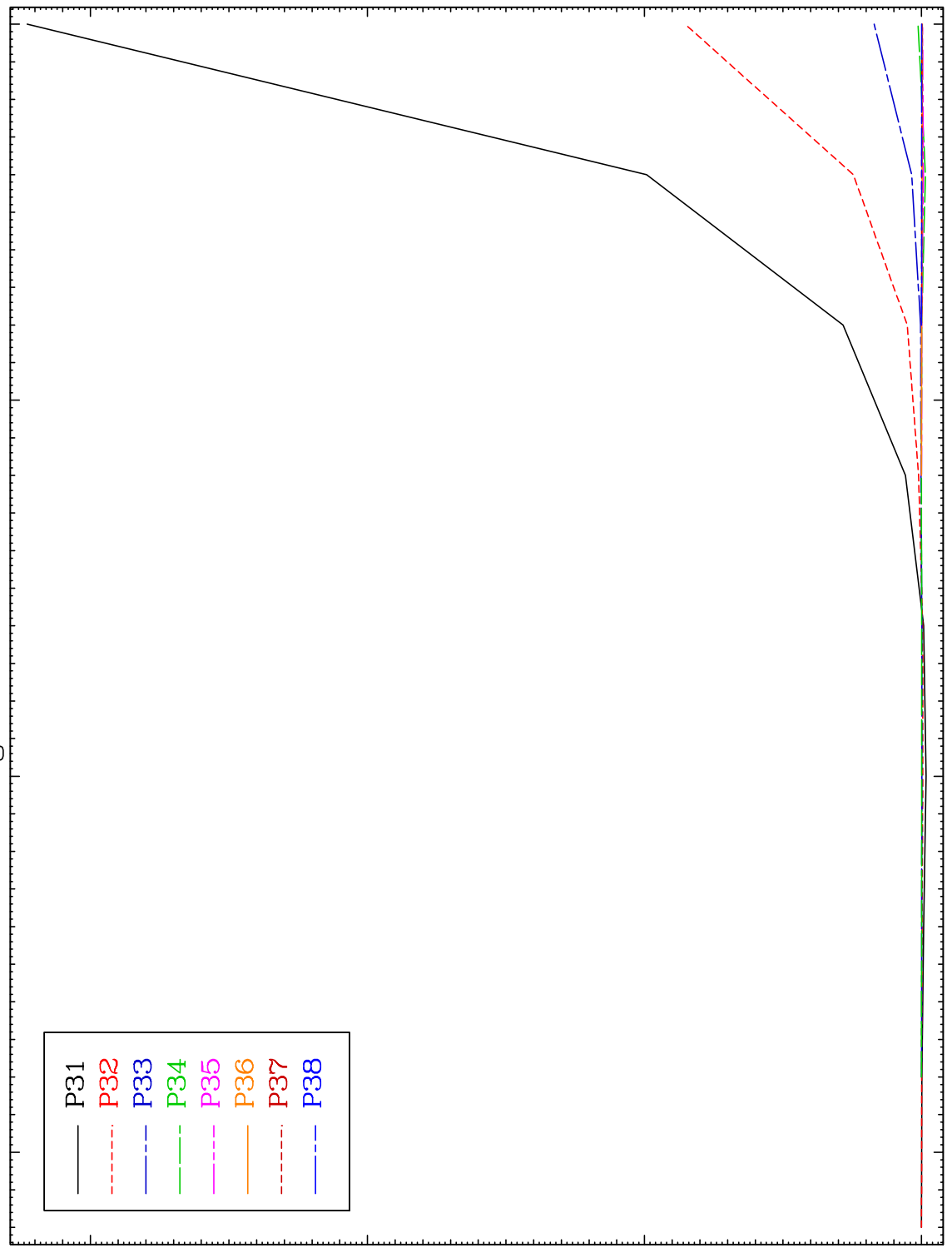
Elastic
Legendre Coefficients

102-No-254



$\times 10^{-6}$

Legendre (CM)

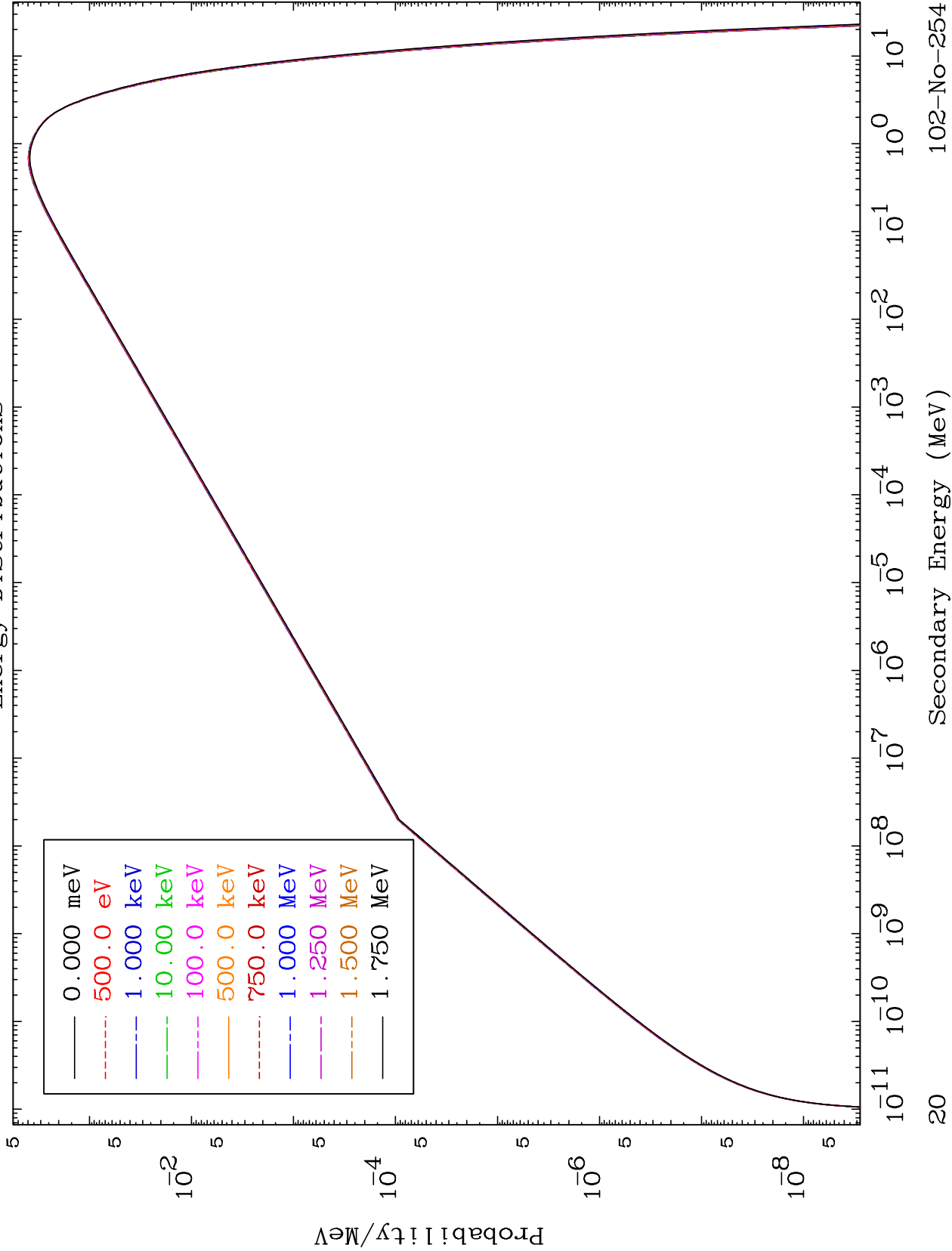


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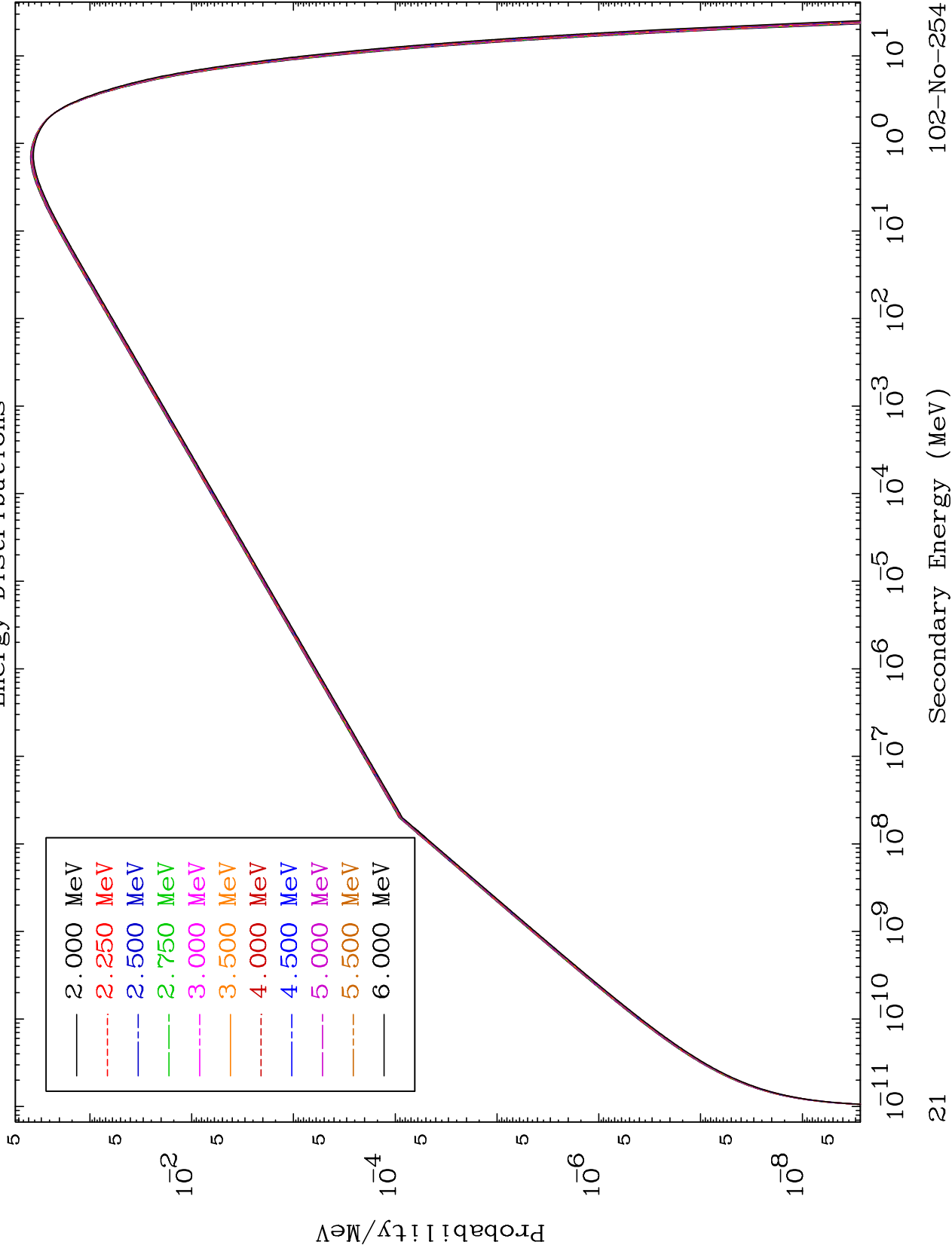
Incident Energy (MeV)

102-No-254

Fission Energy Distributions



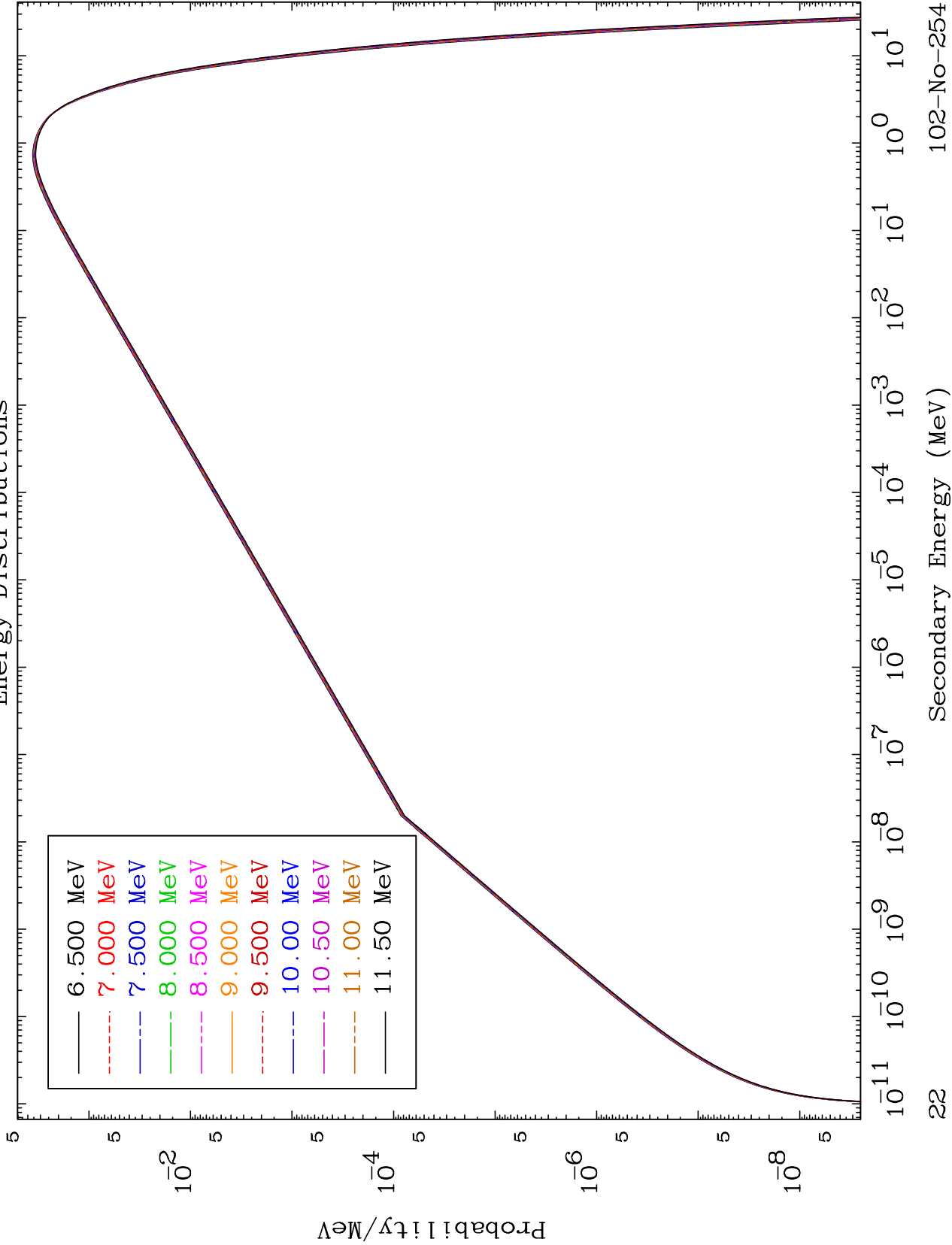
Fission Energy Distributions



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Fission Energy Distributions

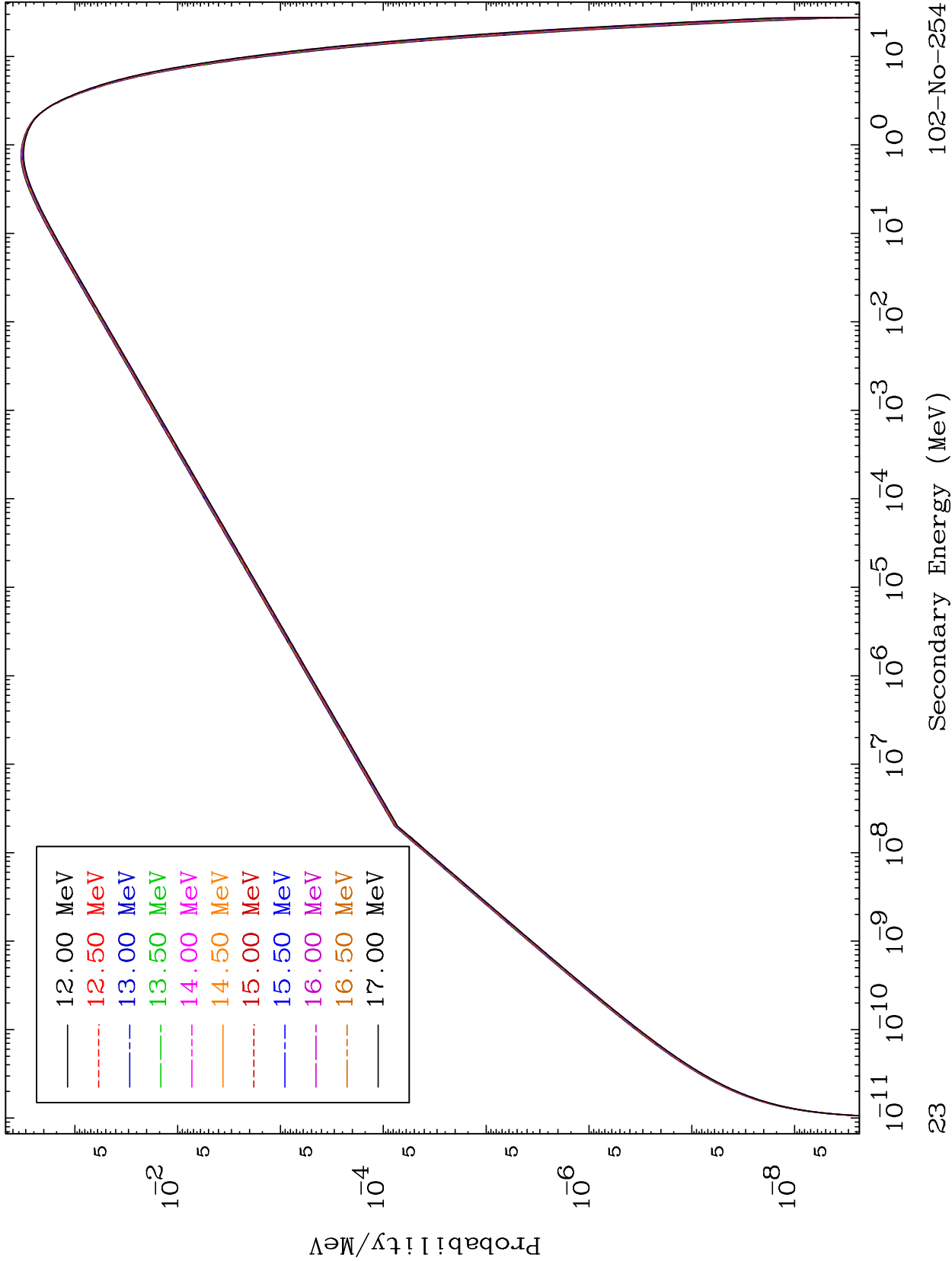
102-No-254



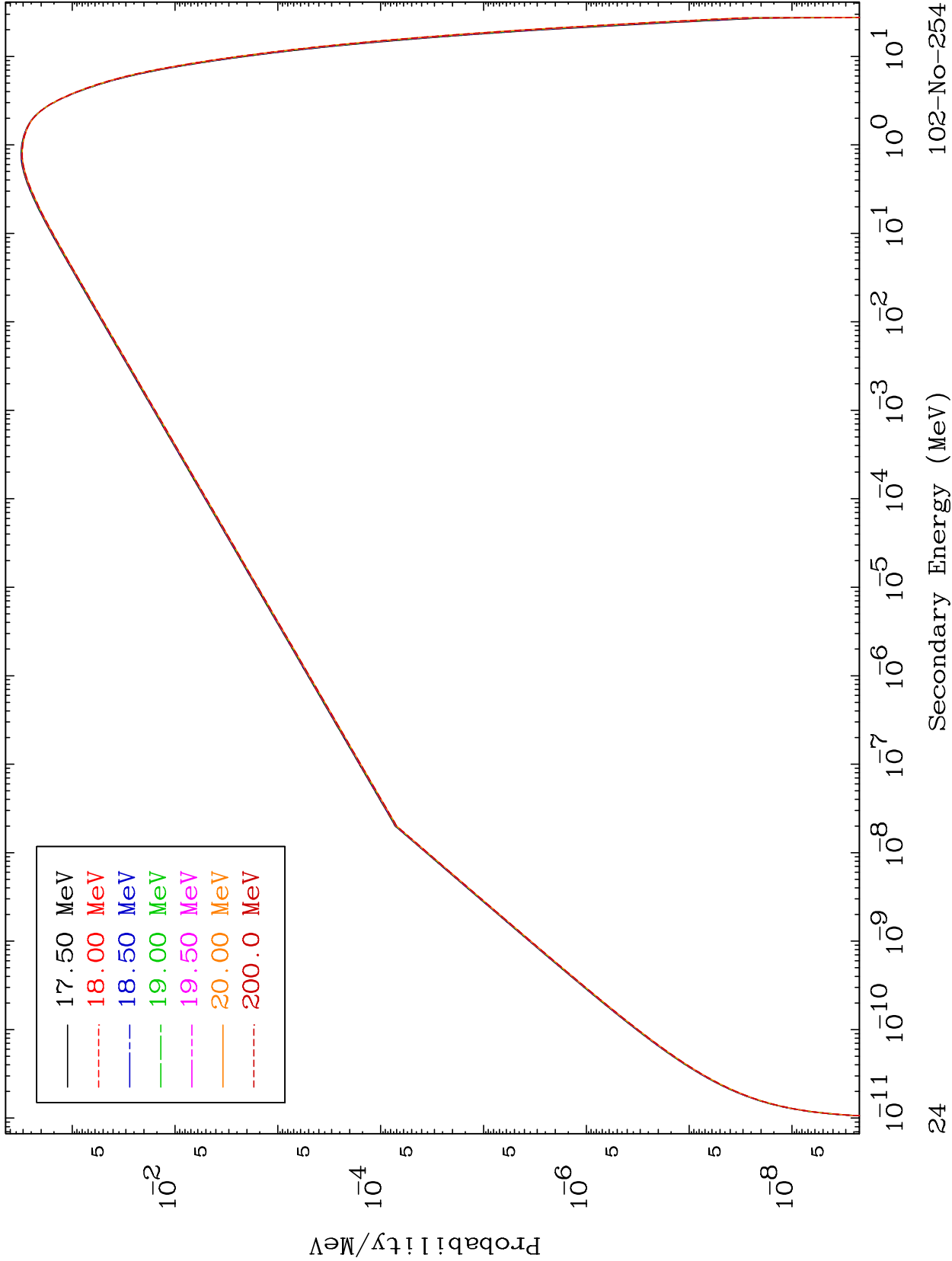
102-No-254

Secondary Energy (MeV)

Fission Energy Distributions



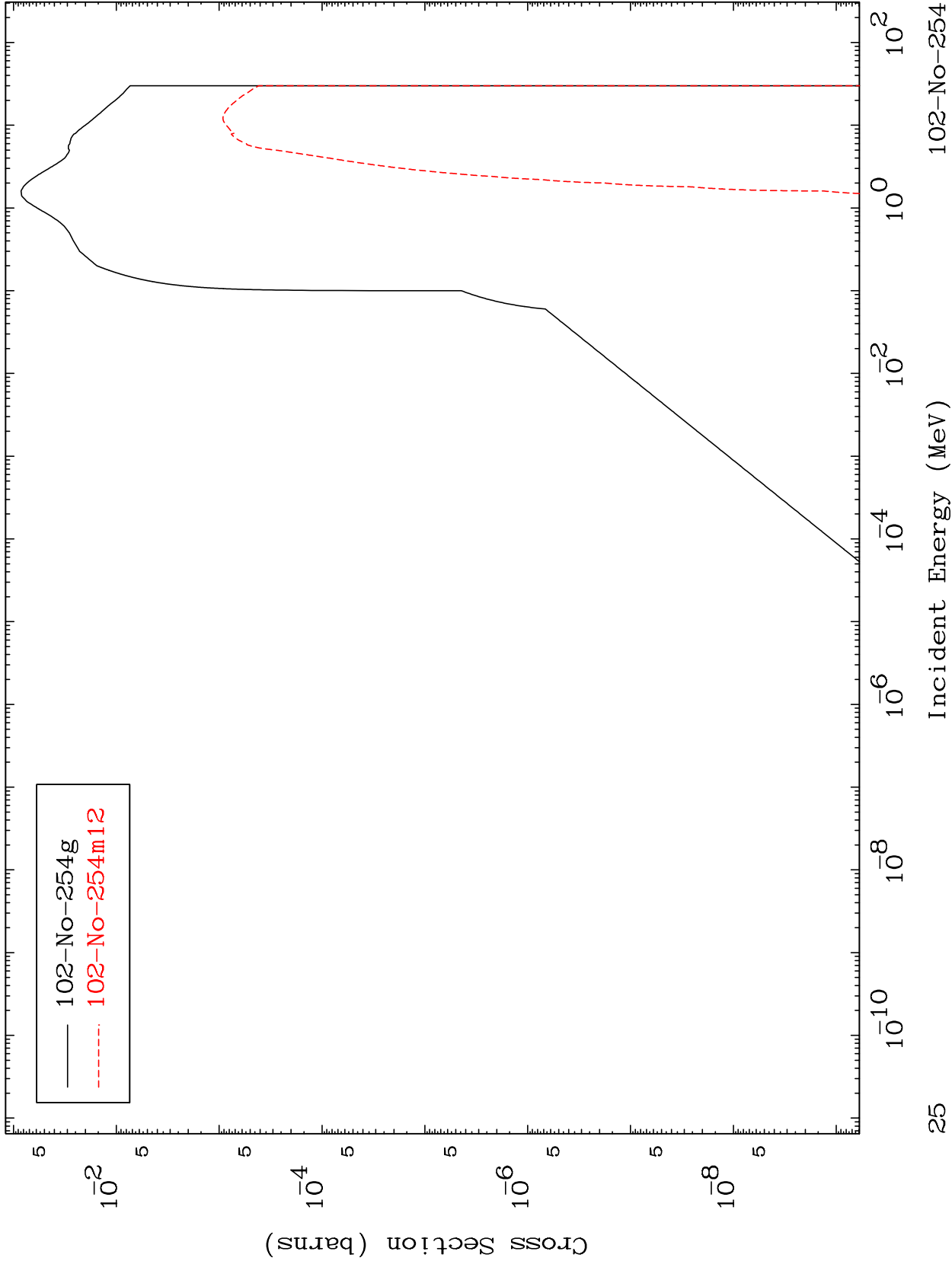
Fission
Energy Distributions



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Radionuclide Production Cross Section

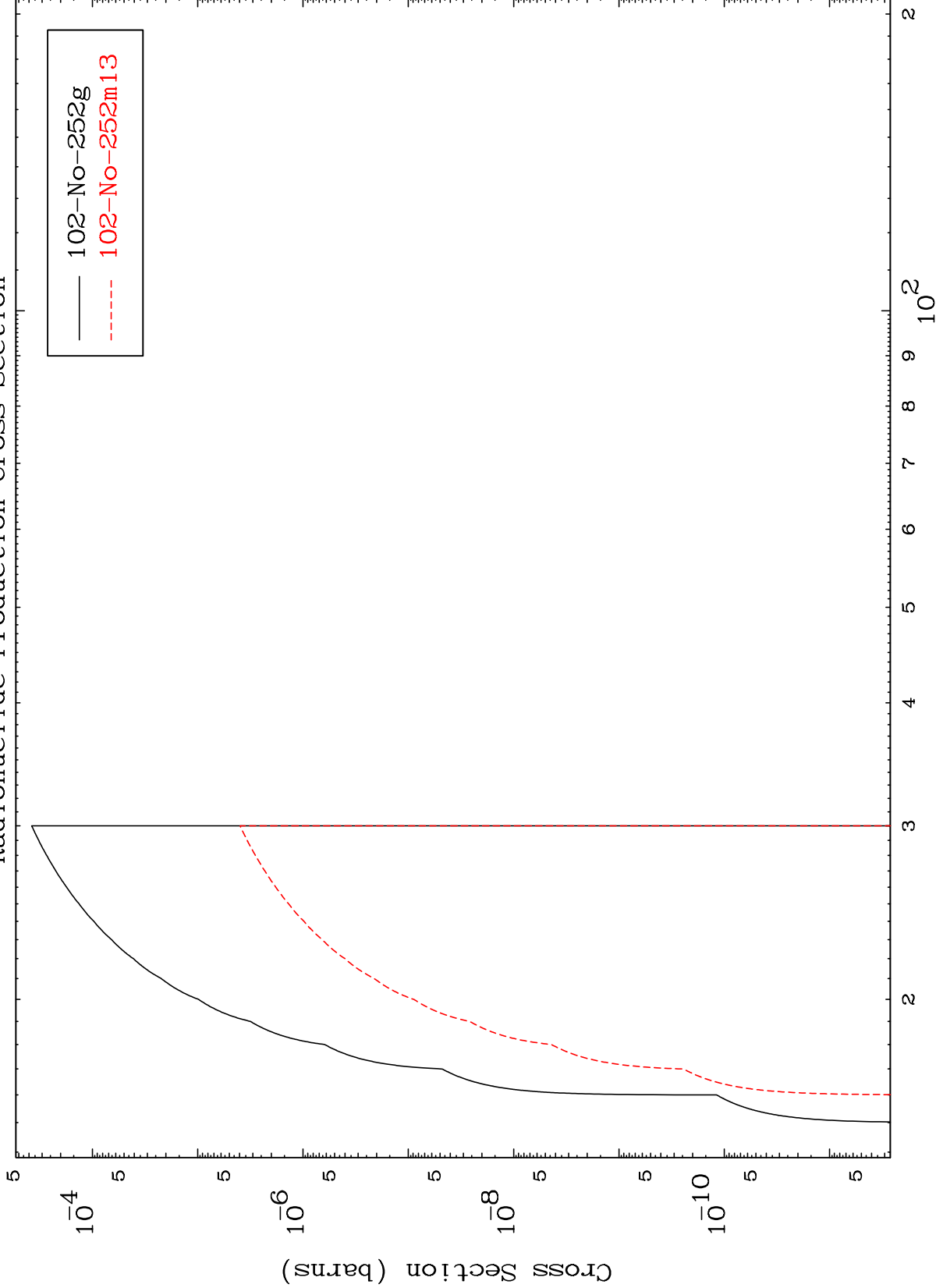
102-No-254



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102-No-254

(n,3n)
Radionuclide Production Cross Section

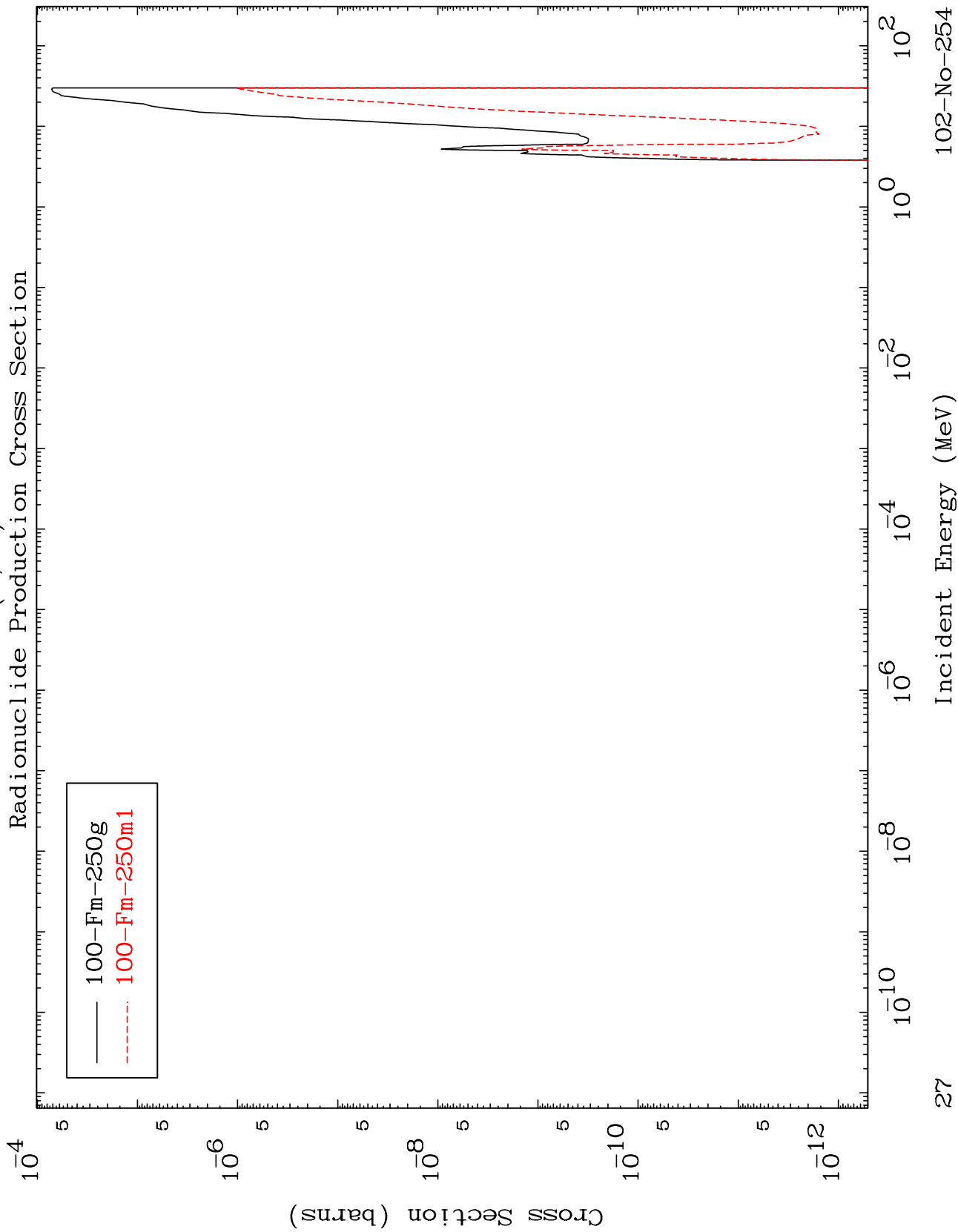


MAT 9972

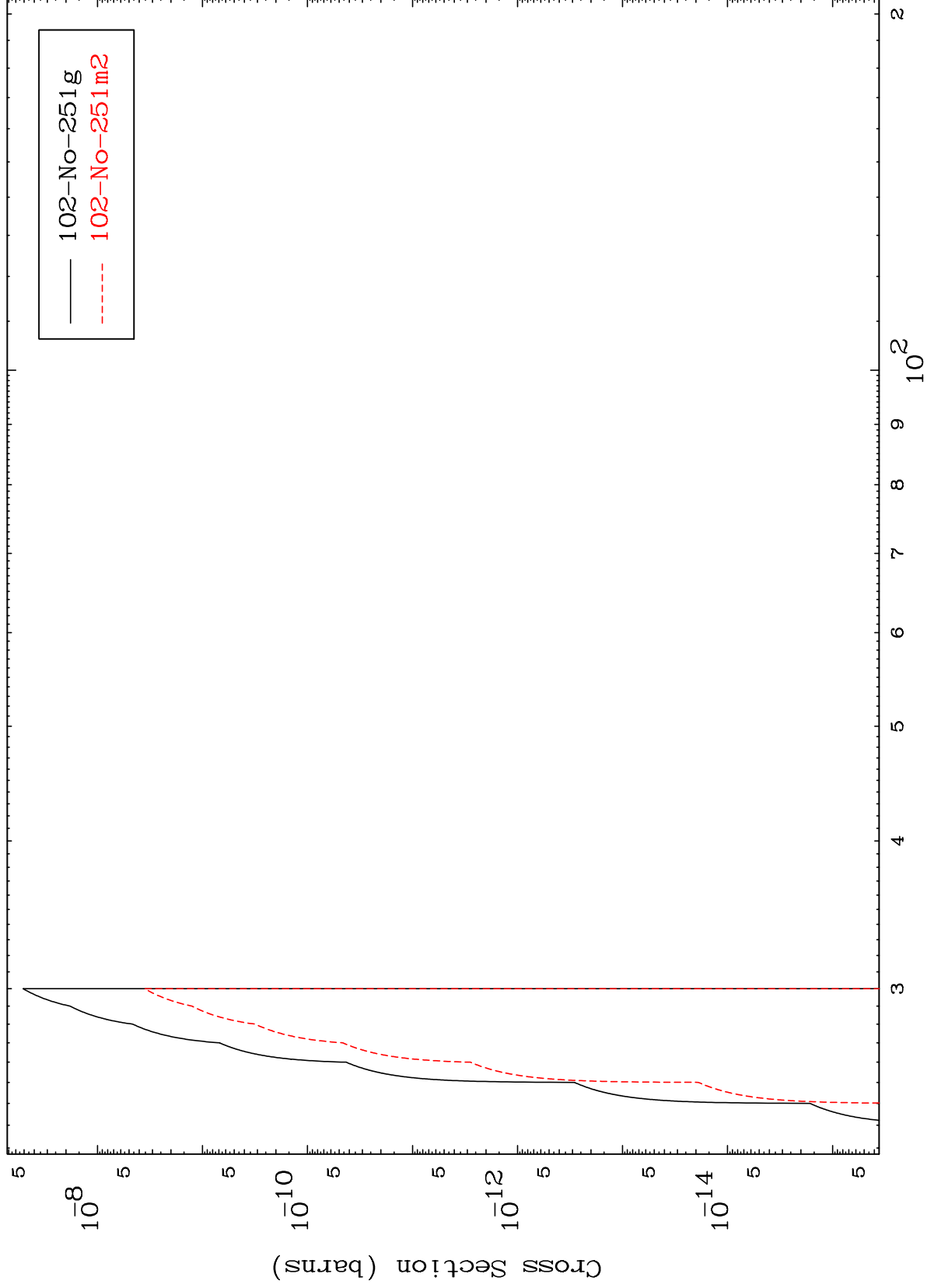
$(n, n') \alpha$

102-No-254

Radionuclide Production Cross Section



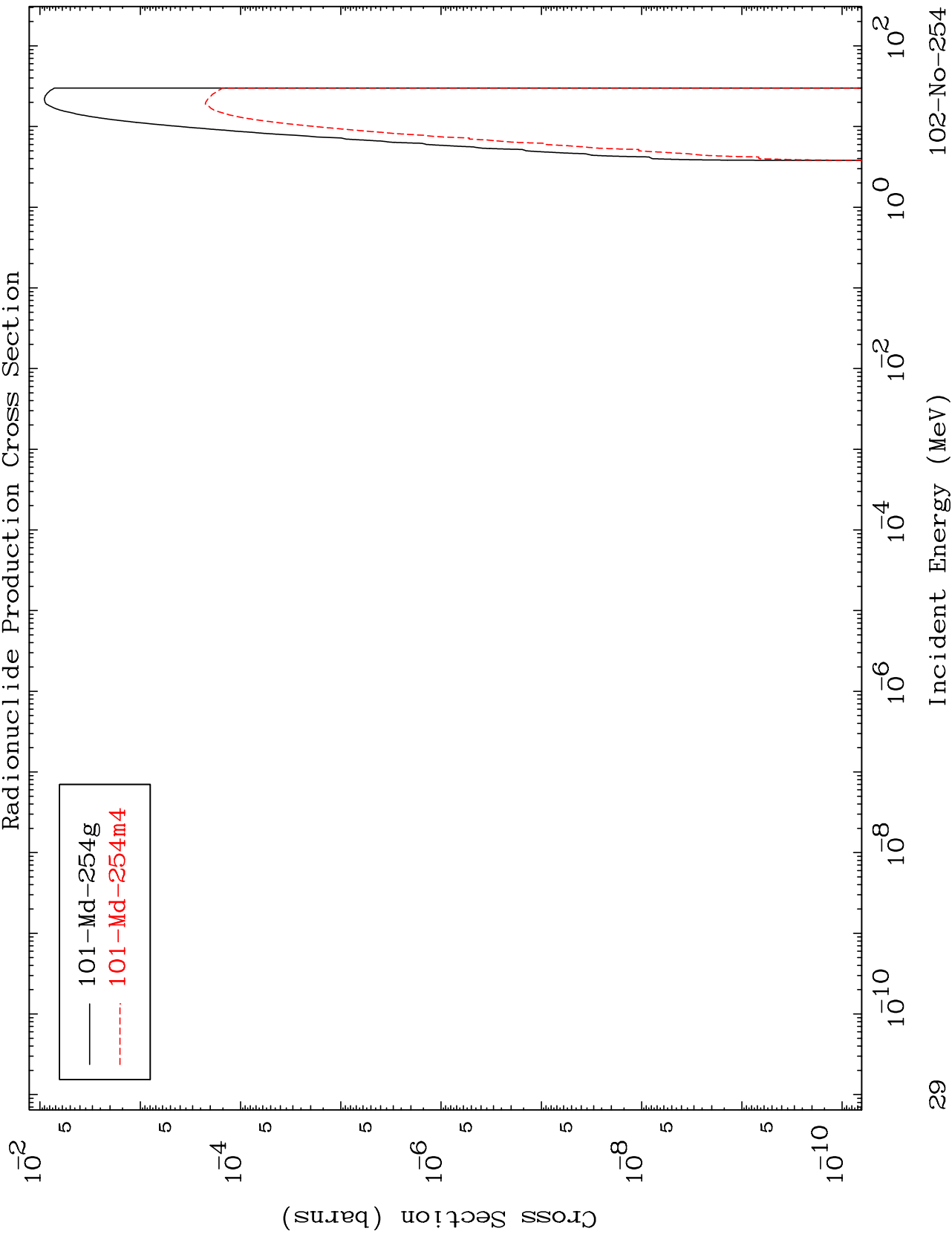
Radionuclide Production Cross Section



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102-No-254

(n,p)
Radionuclide Production Cross Section



101-Md-254g
101-Md-254m4

MAT 9972

(n,p) α

102-No-254

Radionuclide Production Cross Section

