

Program EVALPLOT
(Version 2021-1)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

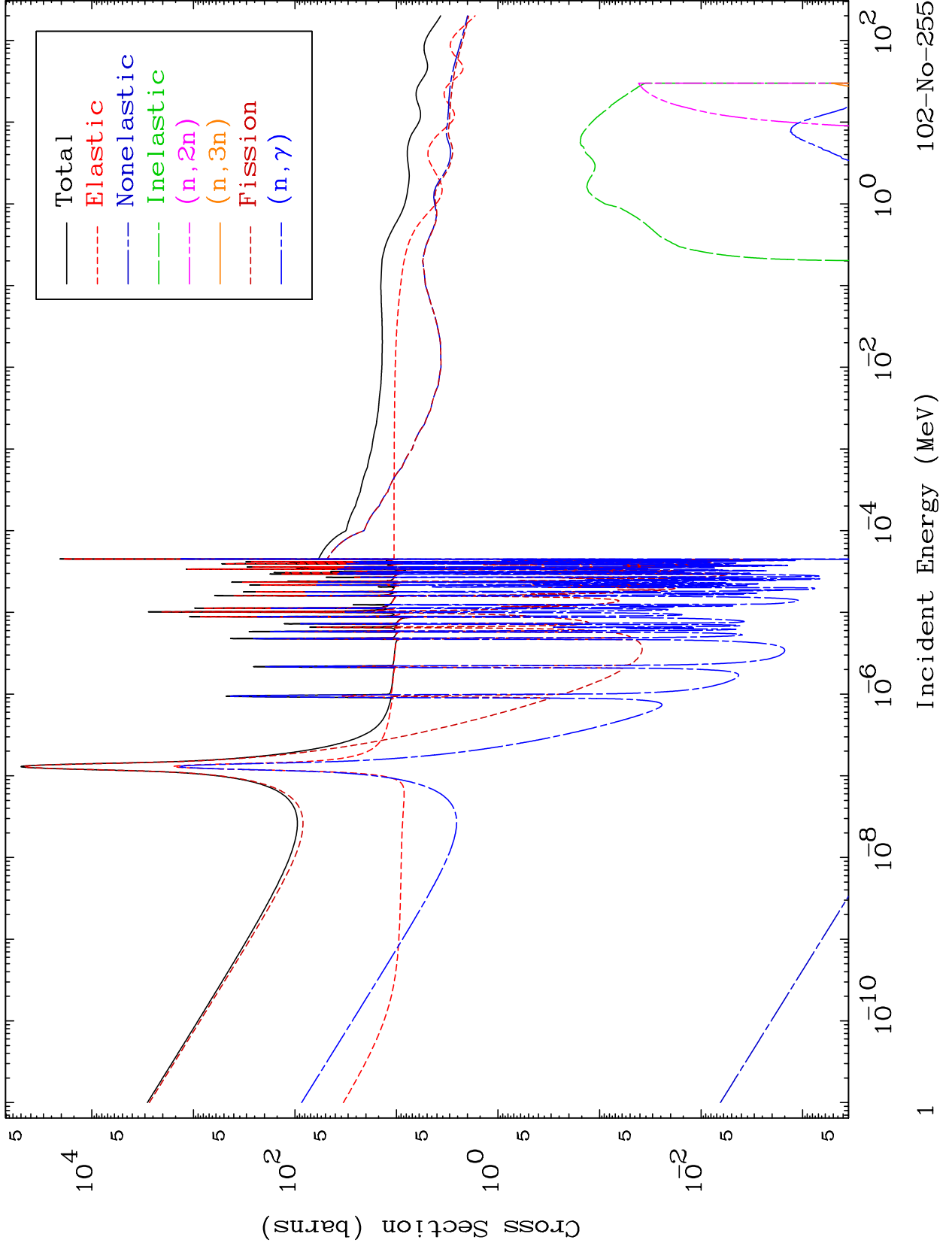
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9973

Neutron Major
293 Kelvin Cross Sections

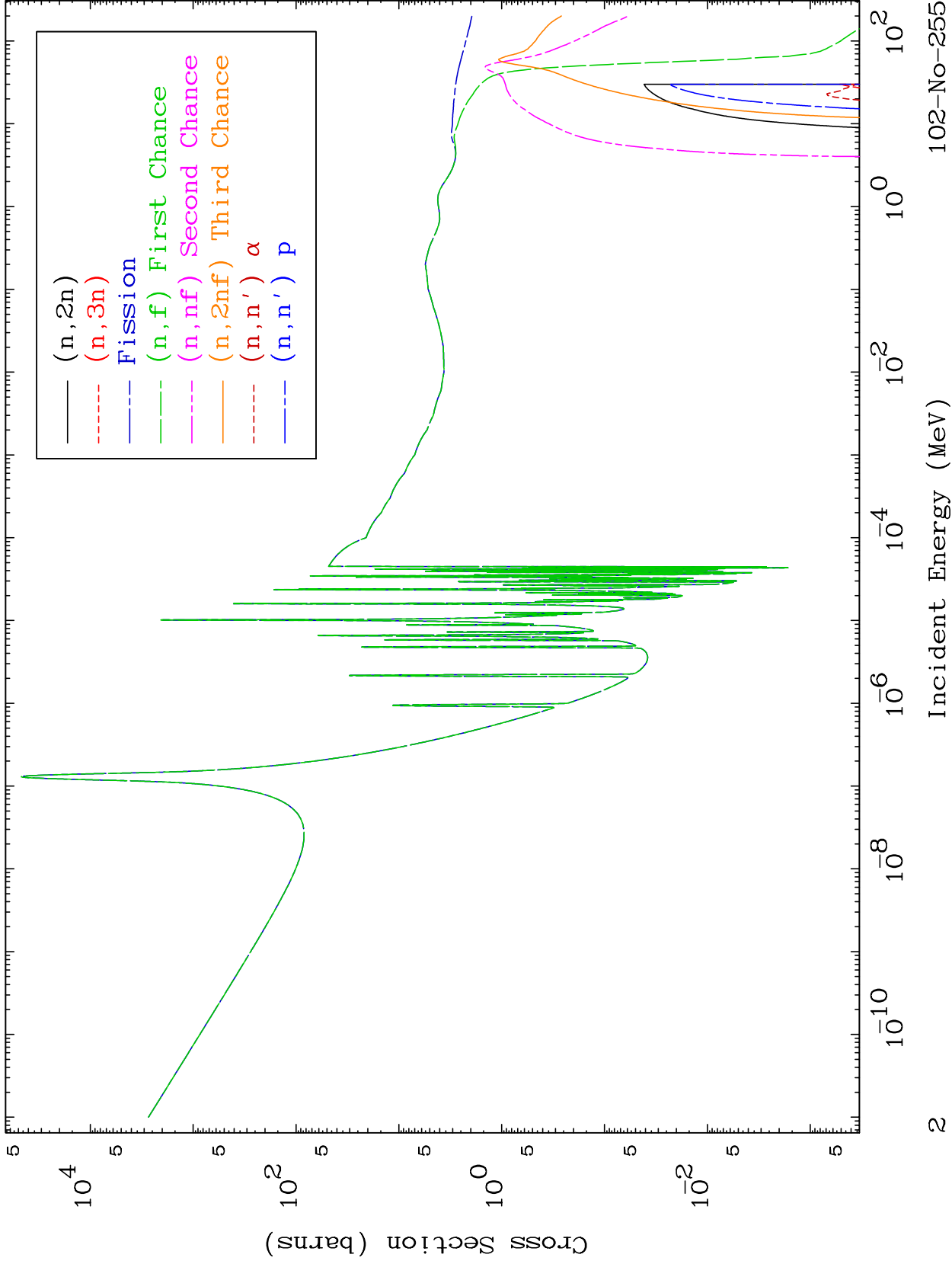
102-No-255



MAT 9973

Neutron Absorption
293 Kelvin Cross Sections

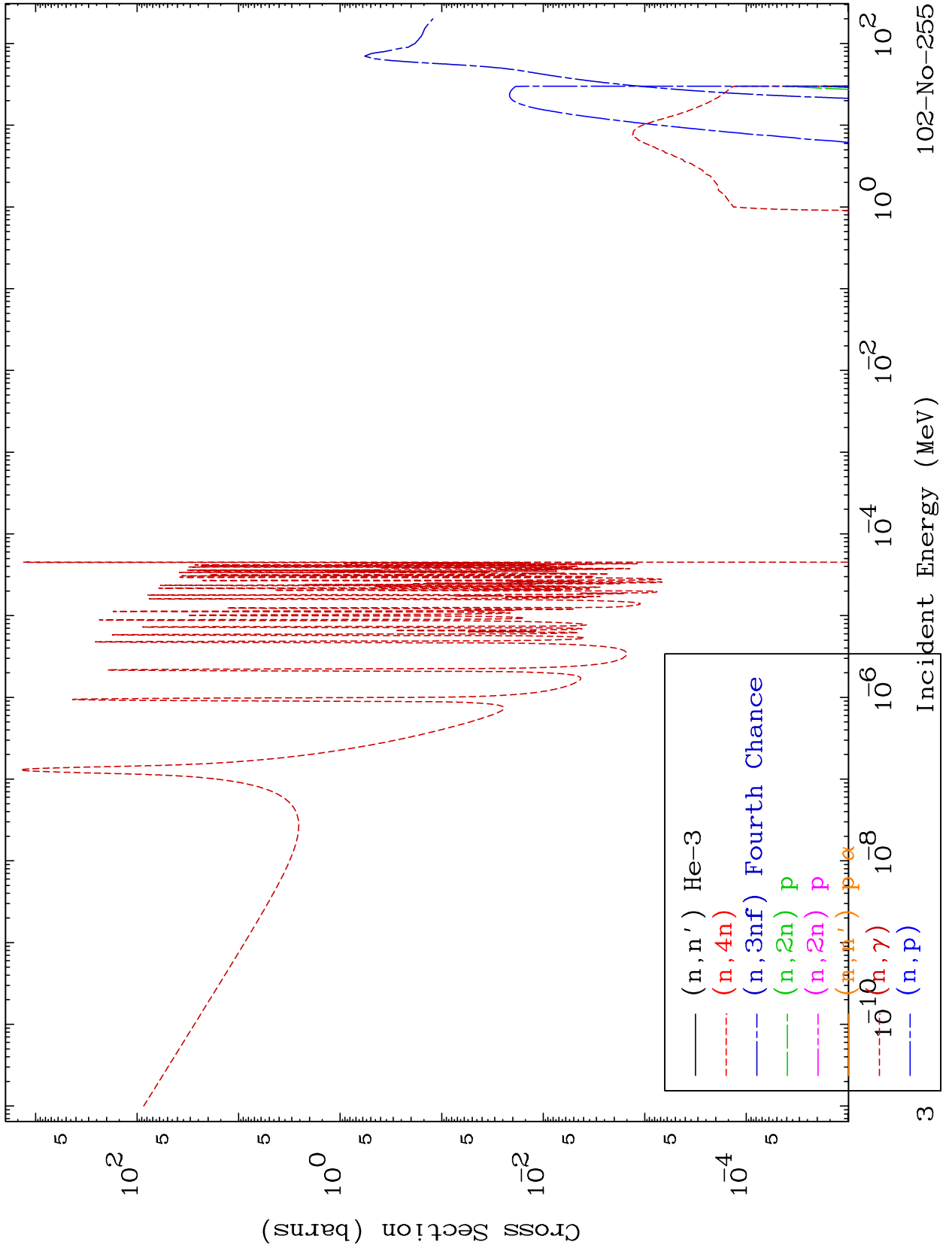
102-No-255



MAT 9973

Neutron Absorption
293 Kelvin Cross Sections

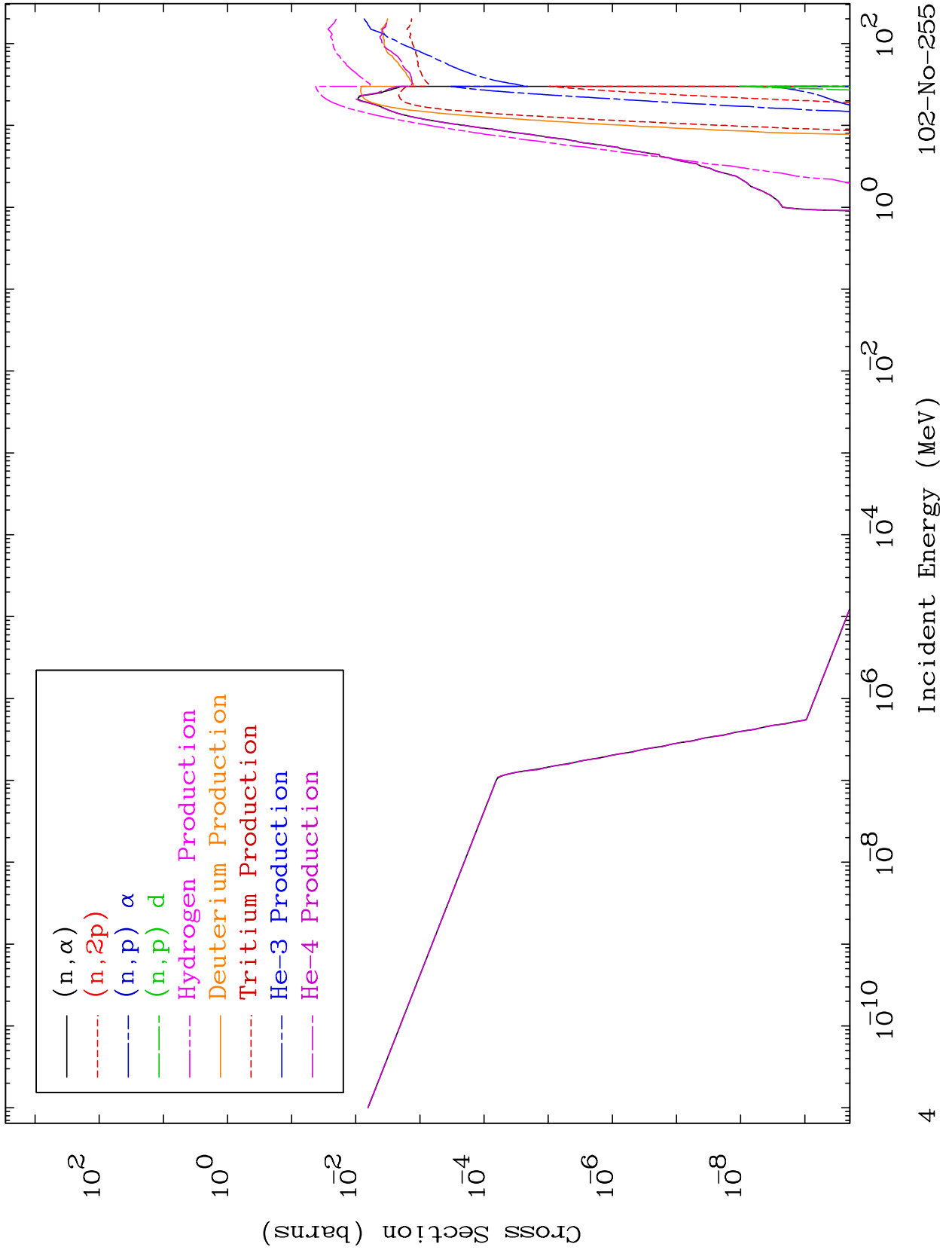
102-No-255



MAT 9973

Neutron Absorption
293 Kelvin Cross Sections

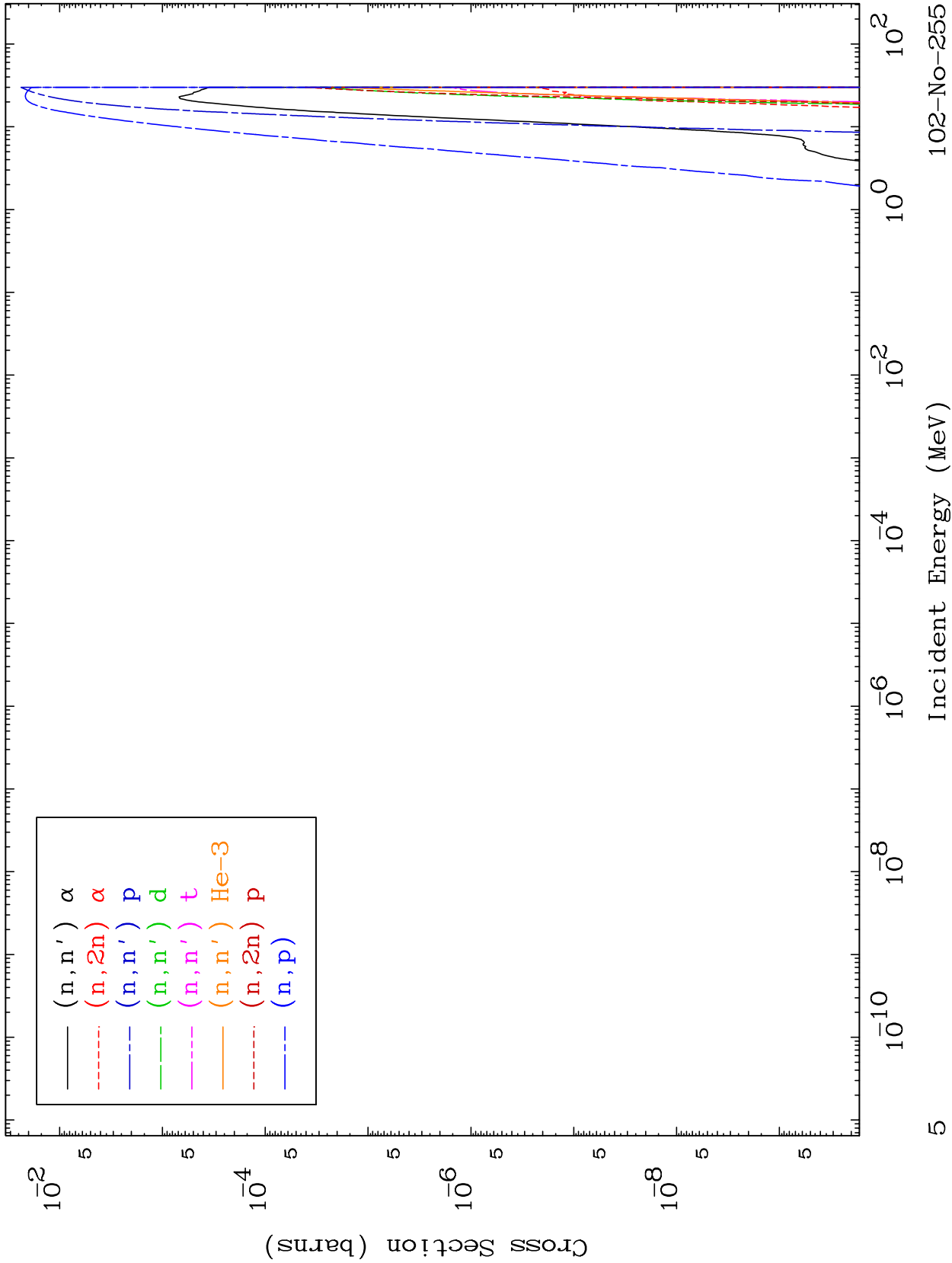
102-No-255

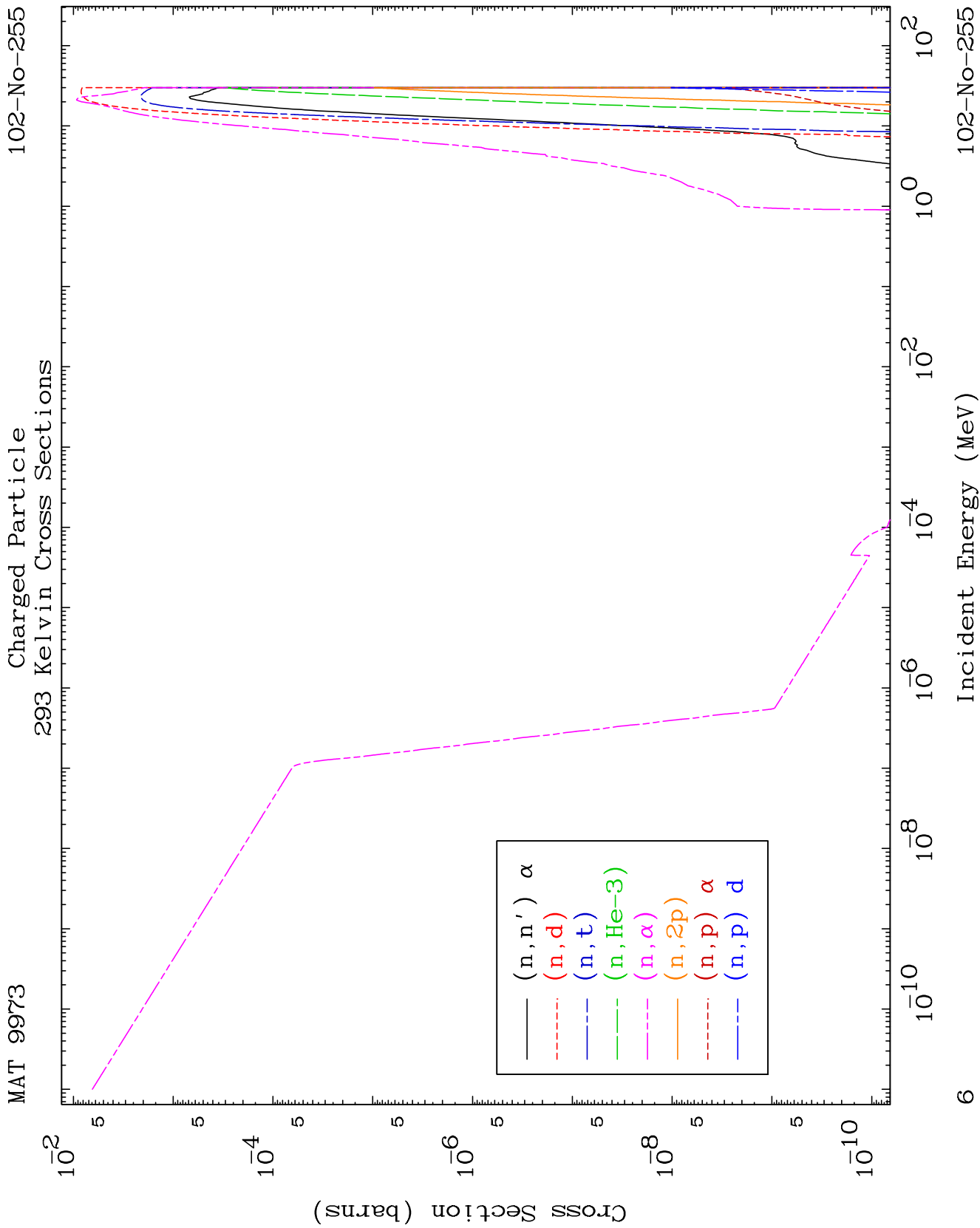


MAT 9973

Charged Particle
293 Kelvin Cross Sections

102-No-255

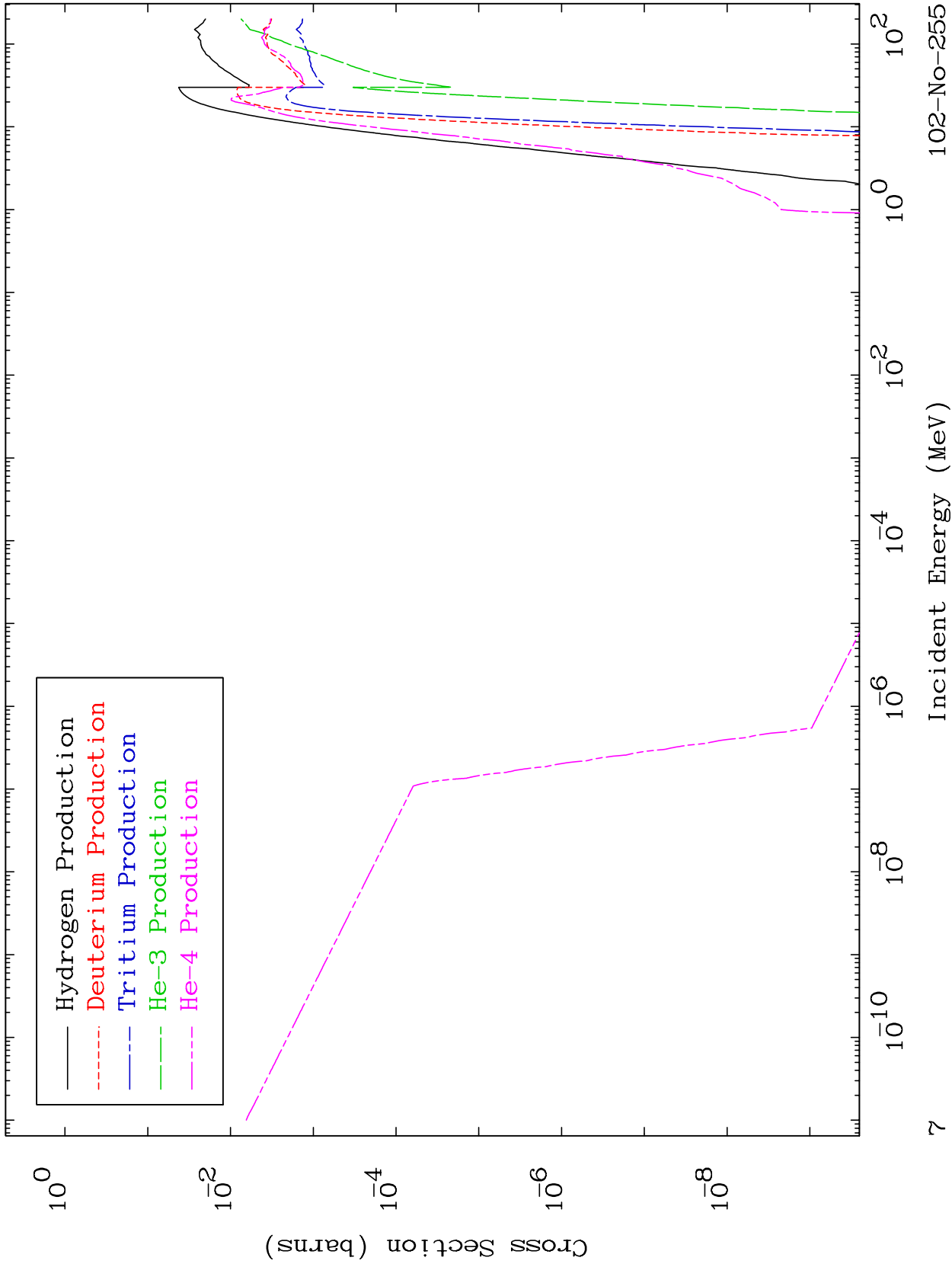




MAT 9973

Particle Production
293 Kelvin Cross Sections

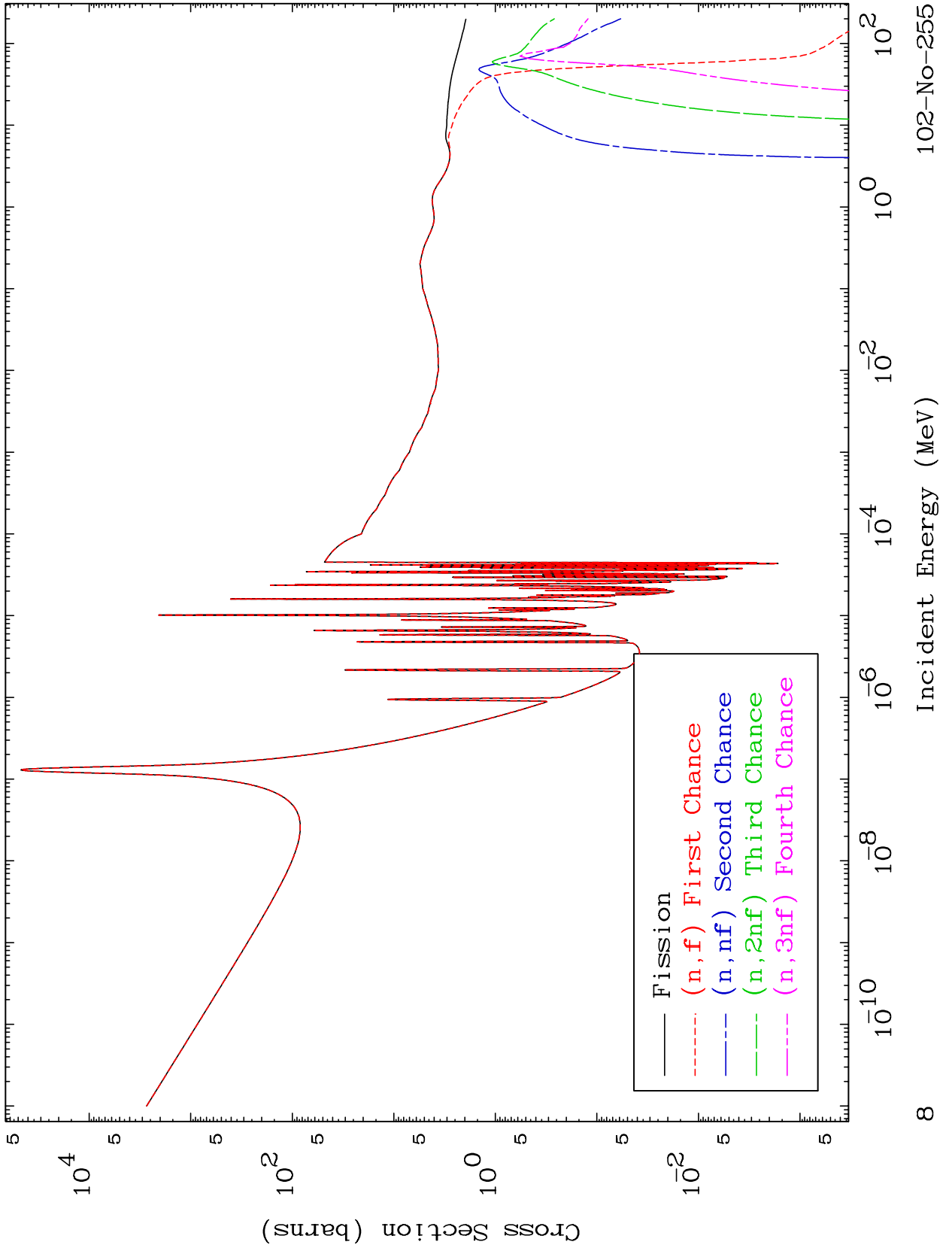
102-No-255



MAT 9973

Fission
293 Kelvin Cross Sections

102-No-255

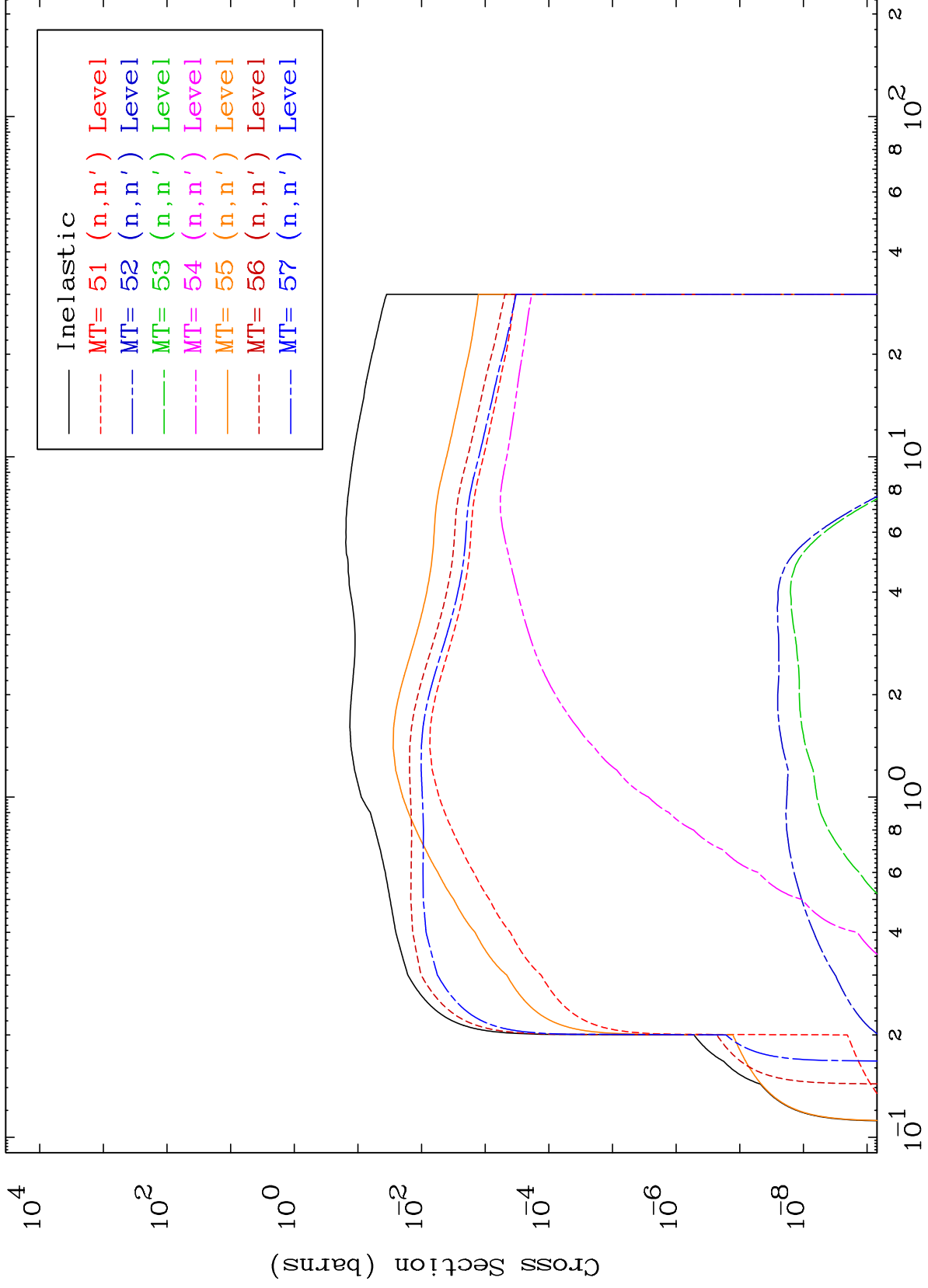


MAT 9973

(n,n') Levels

102-No-255

293 Kelvin Cross Sections



Incident Energy (MeV)

102-No-255

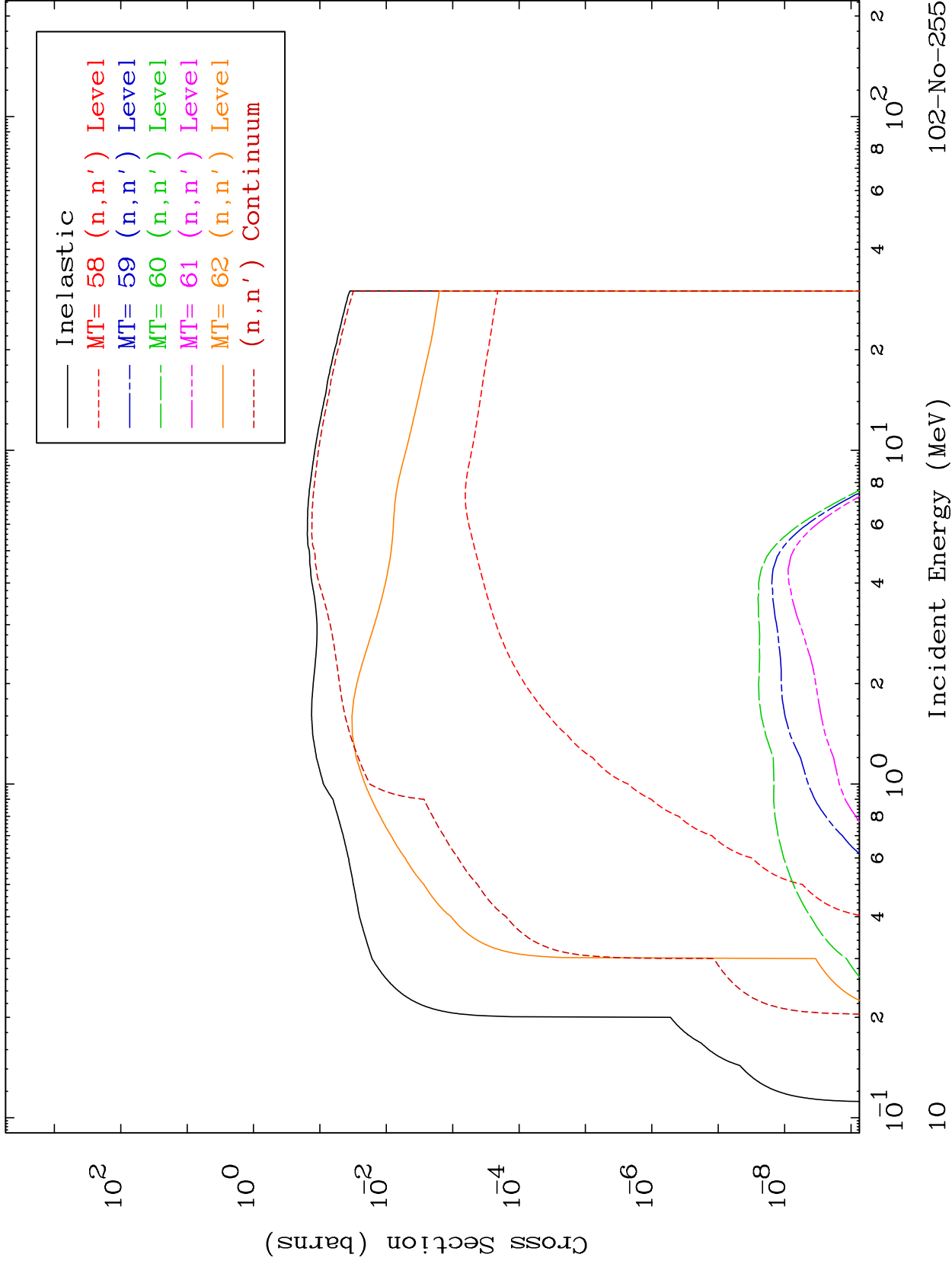
9

MAT 9973

(n,n') Levels

102-No-255

293 Kelvin Cross Sections



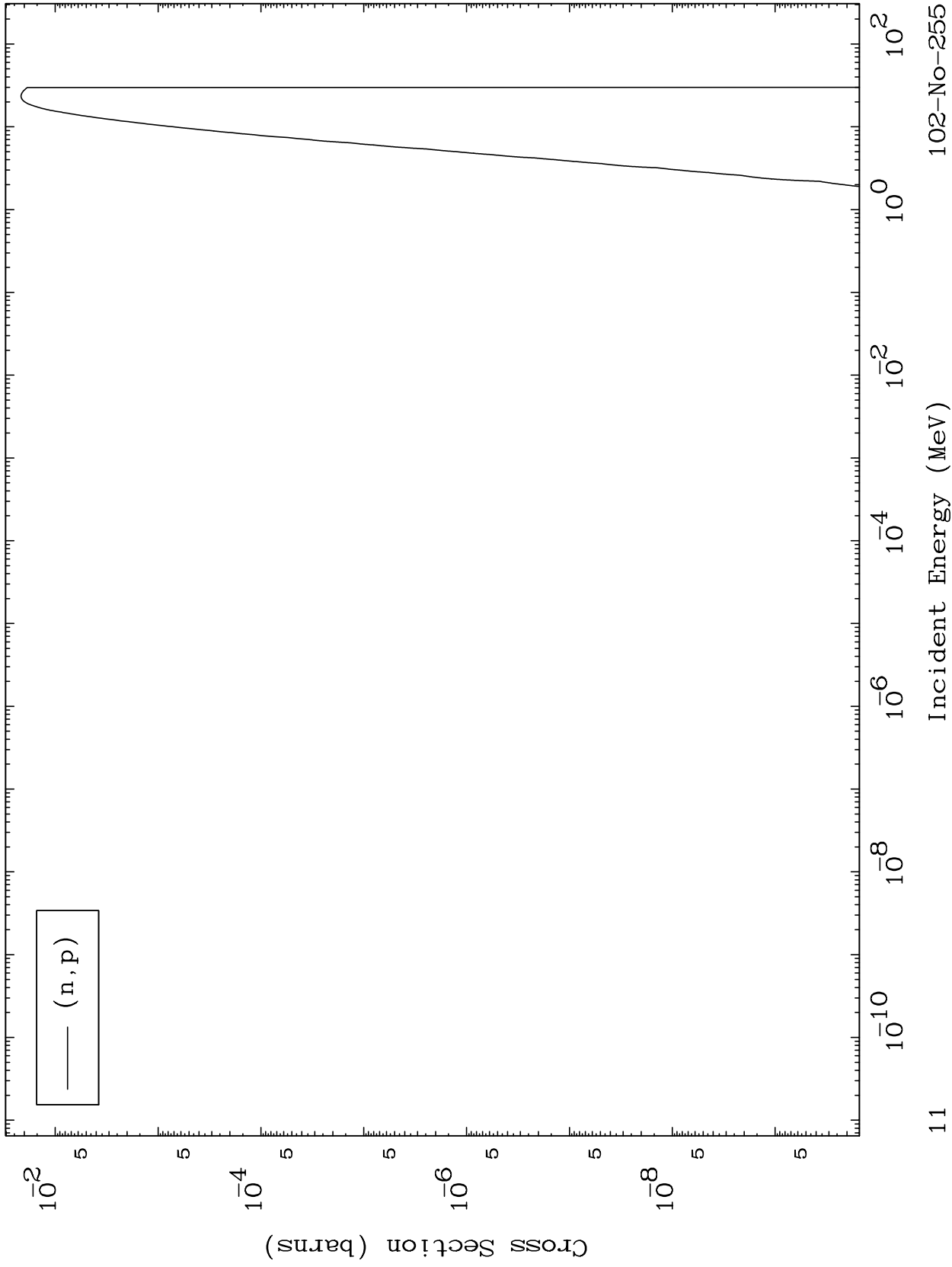
102-No-255

Incident Energy (MeV)

MAT 9973

(n,p) Levels
293 Kelvin Cross Sections

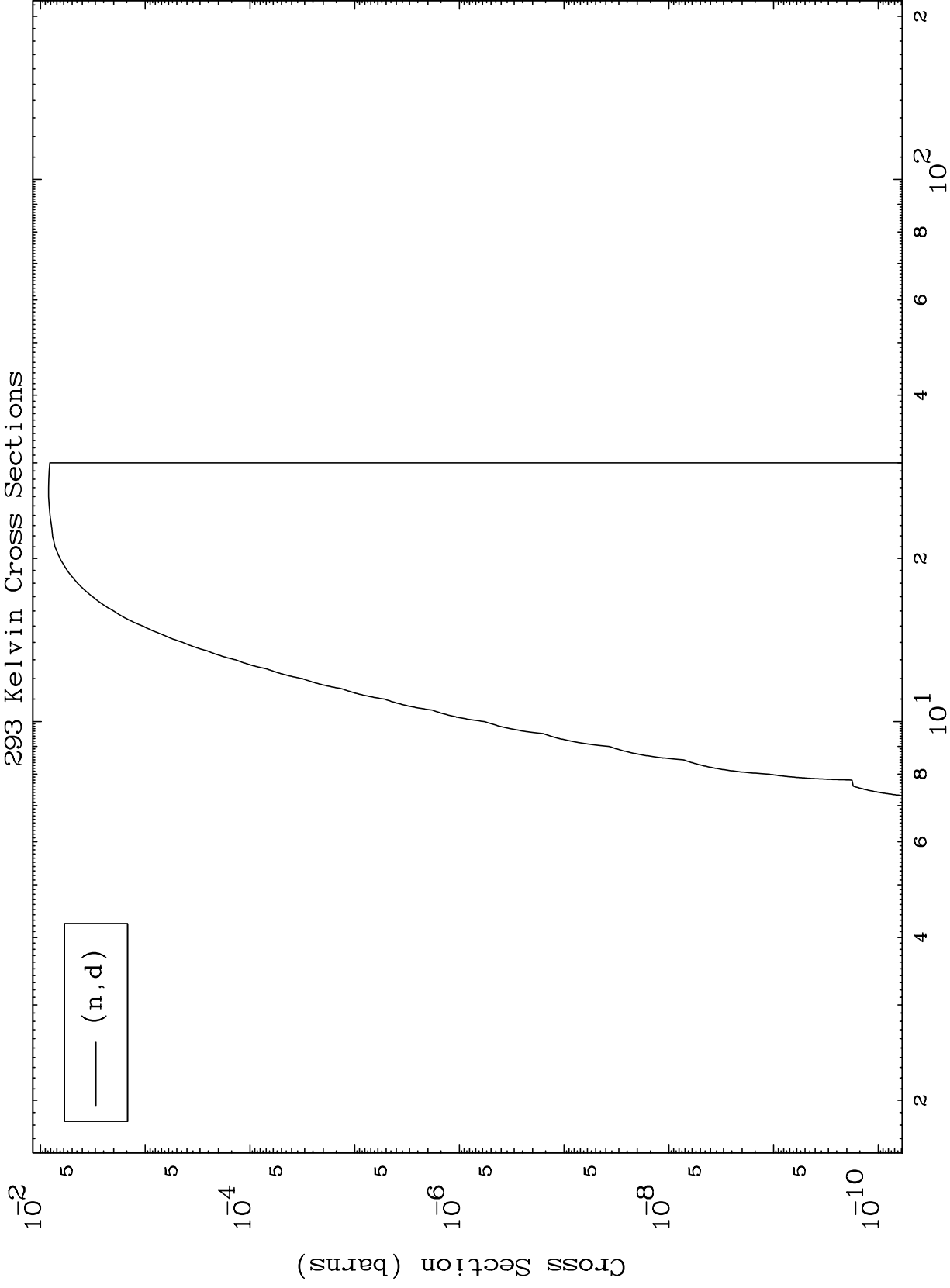
102-No-255



MAT 9973

(n,d) Levels
293 Kelvin Cross Sections

102-No-255



12

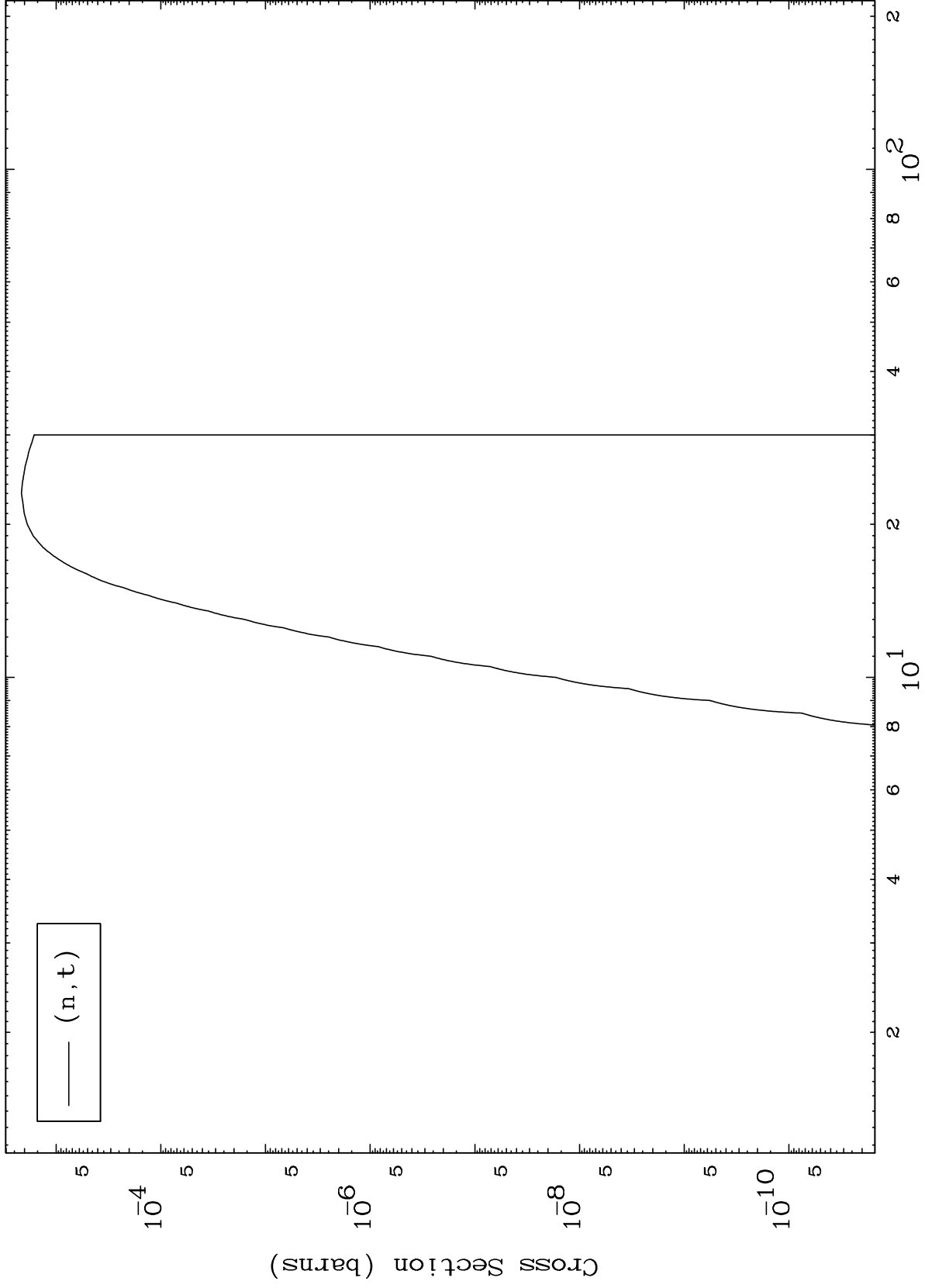
Incident Energy (MeV)

102-No-255

MAT 9973

(n,t) Levels
293 Kelvin Cross Sections

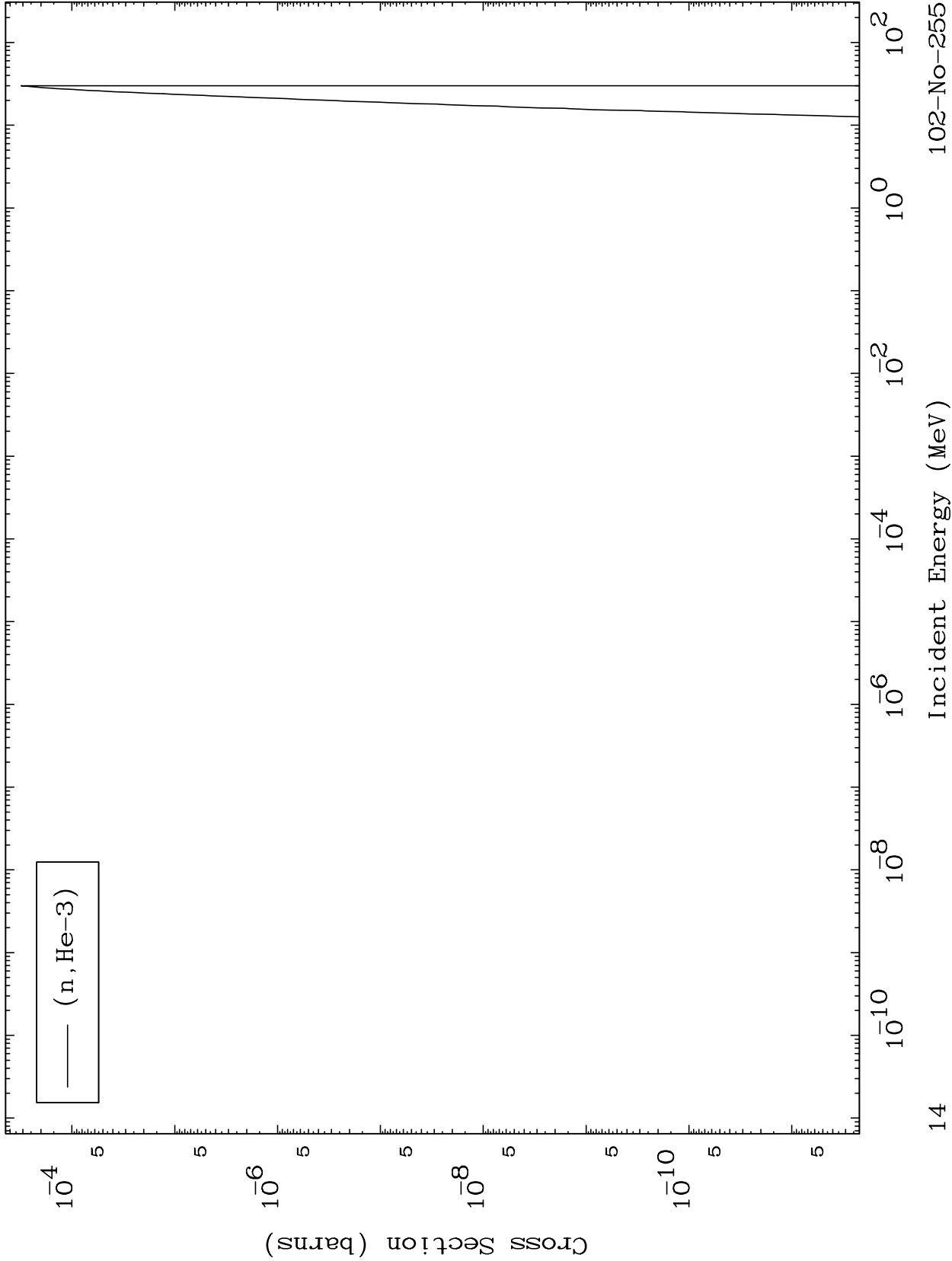
102-No-255



MAT 9973

(n,He3) Levels
293 Kelvin Cross Sections

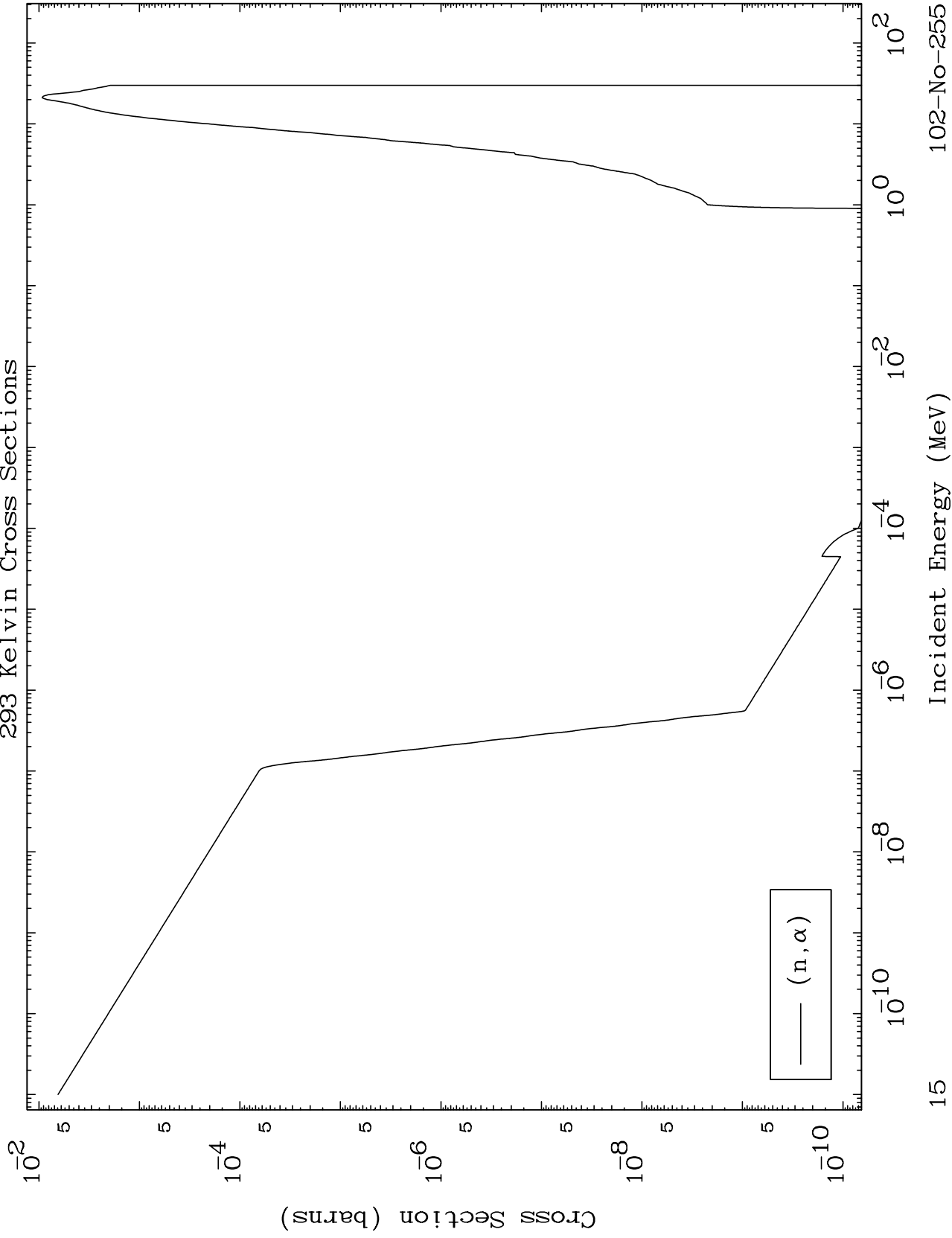
102-No-255



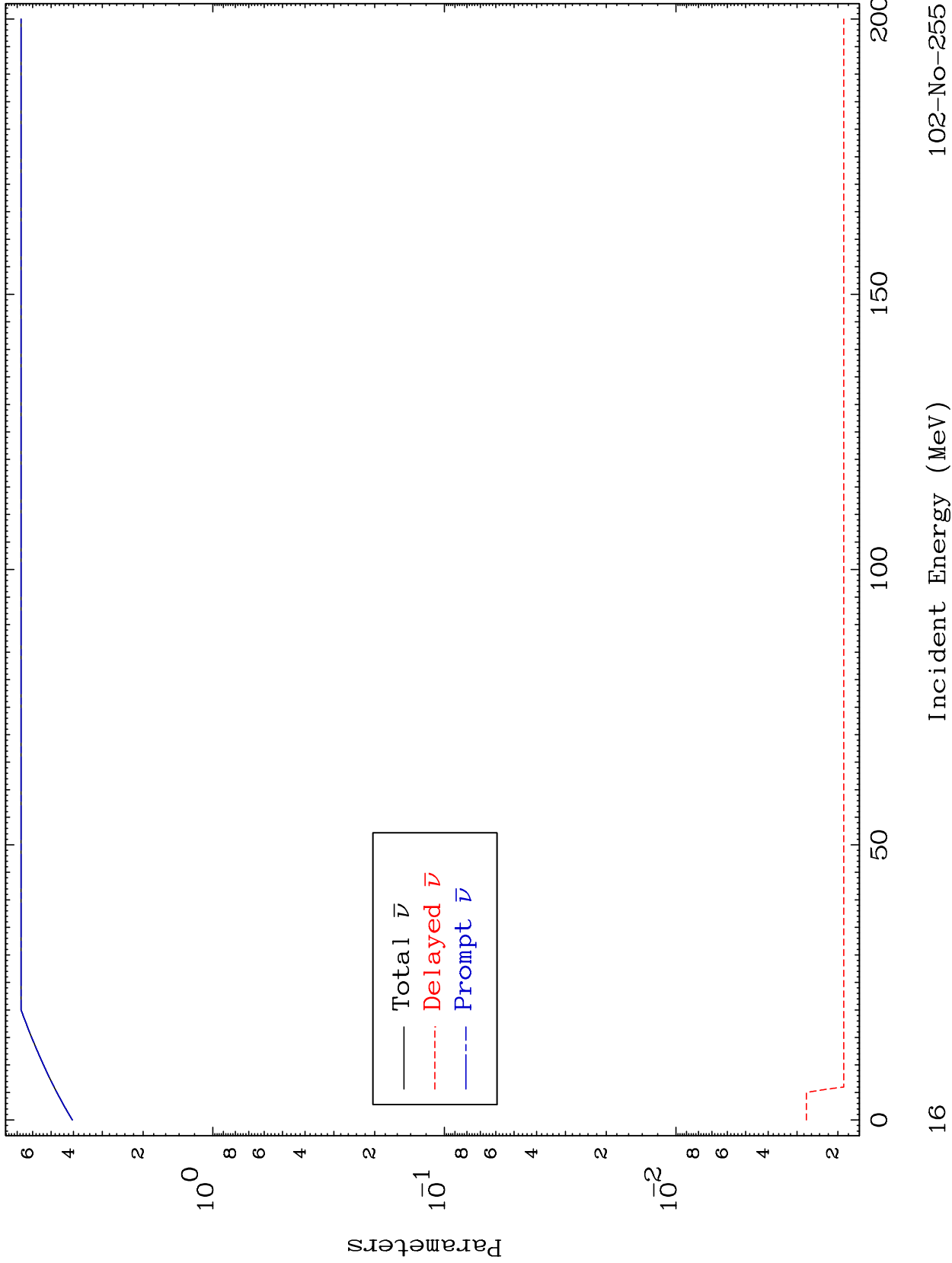
MAT 9973

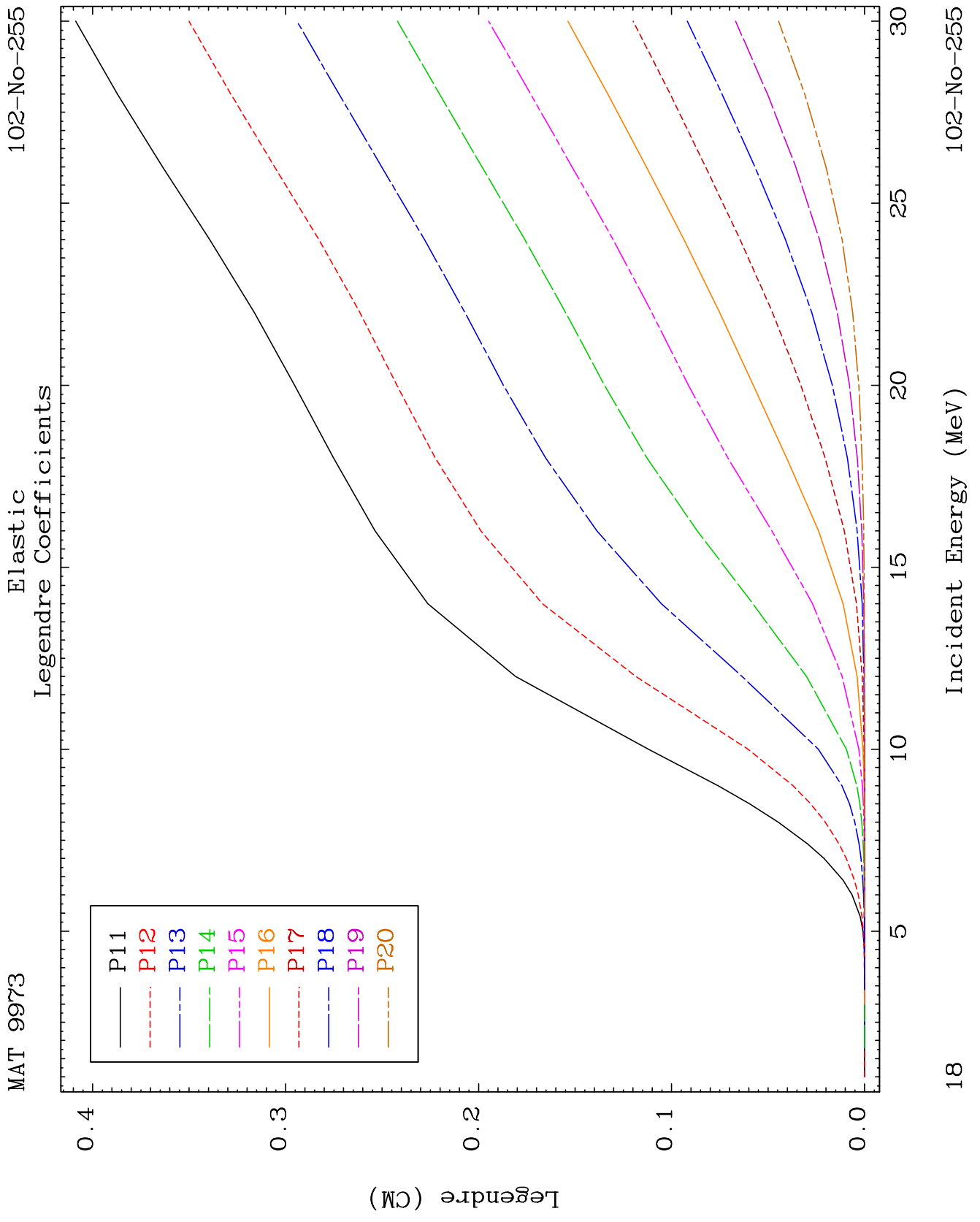
(n, α) Levels
293 Kelvin Cross Sections

102-No-255



(n, α)

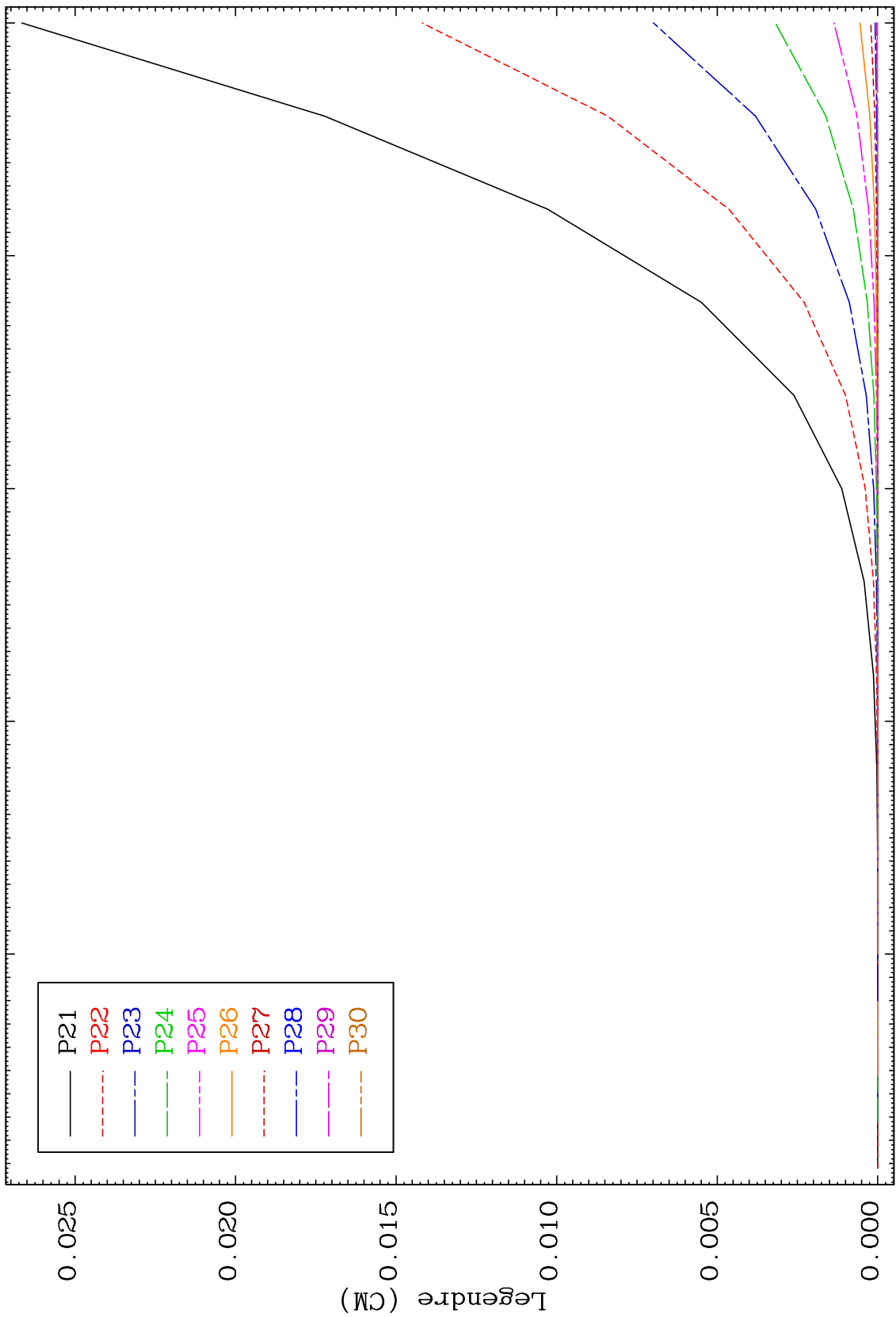




MAT 9973

Elastic Legendre Coefficients

102-No-255



19

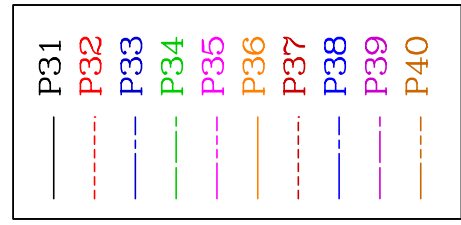
Incident Energy (MeV)

102-No-255

MAT 9973

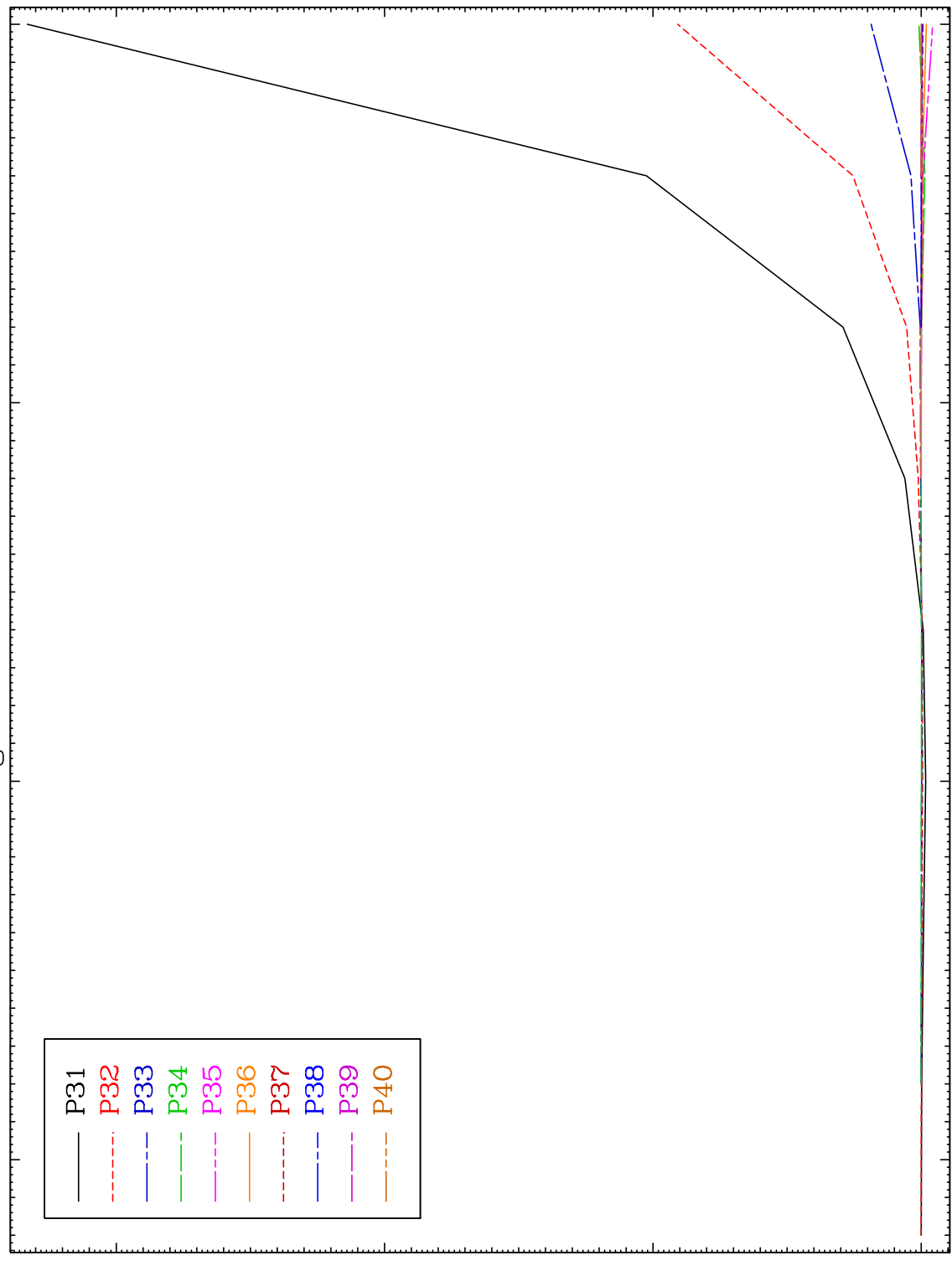
Elastic Legendre Coefficients

102-No-255



$\times 10^{-6}$

Legendre (CM)



30

25

20

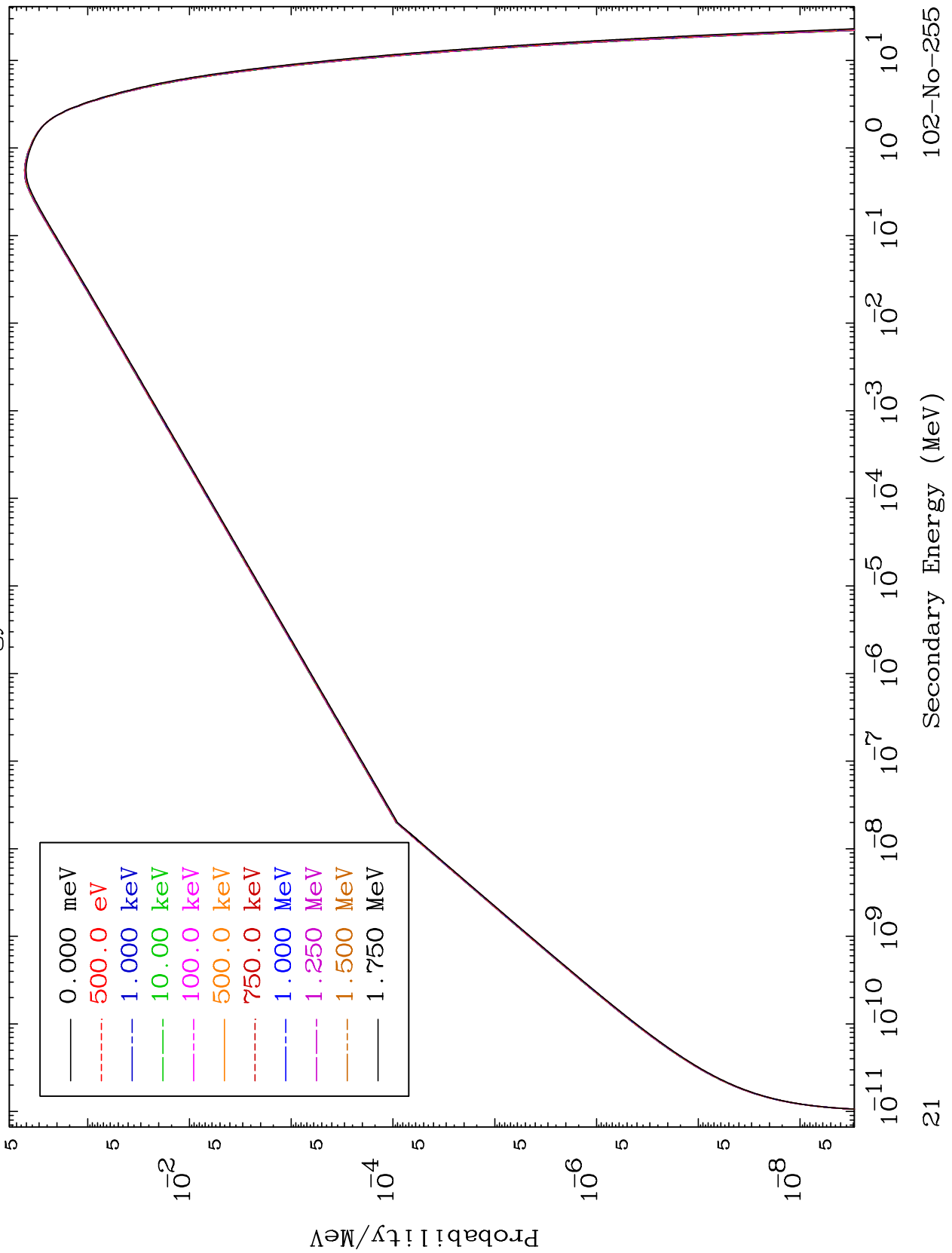
15

Incident Energy (MeV)

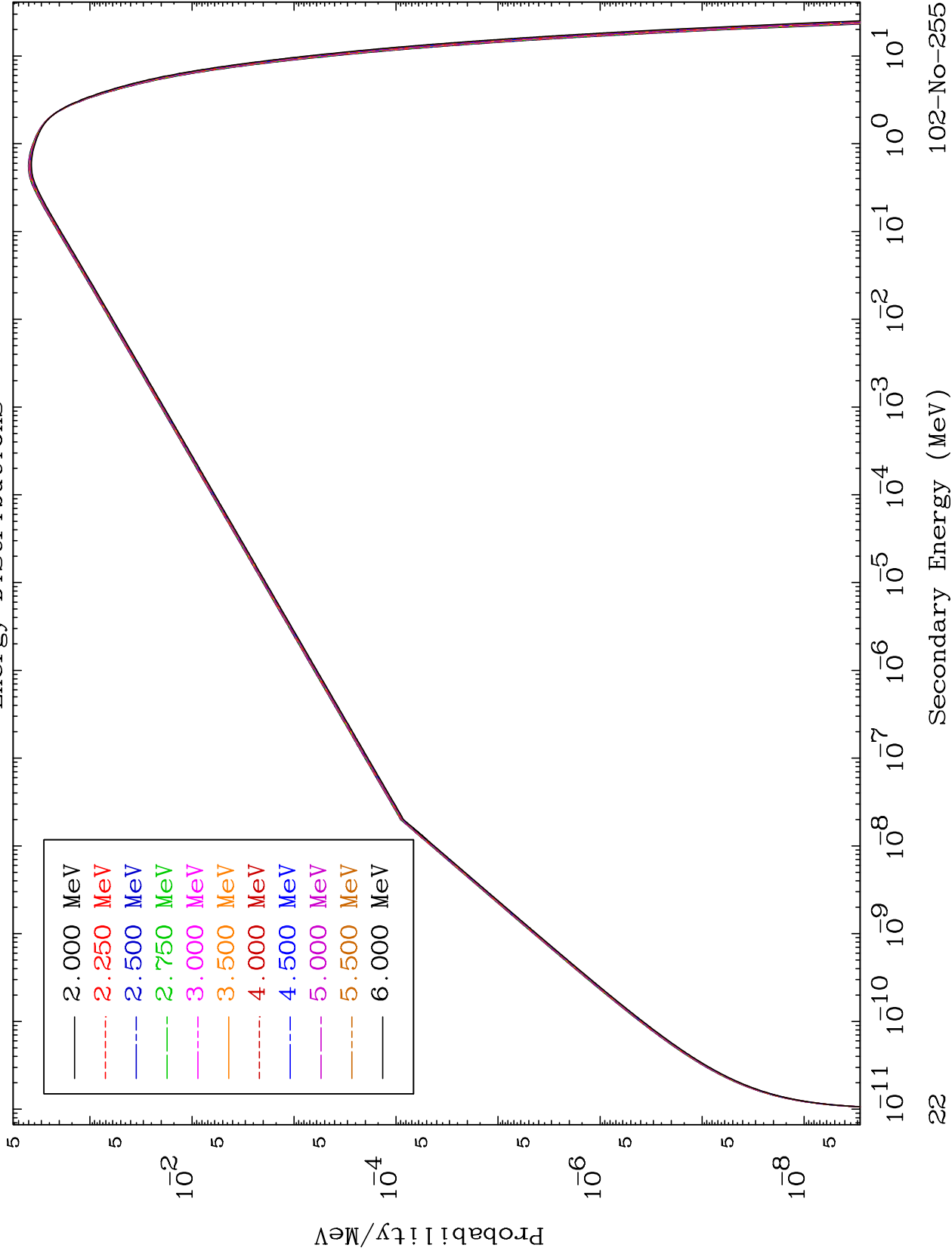
102-No-255

20

Fission Energy Distributions



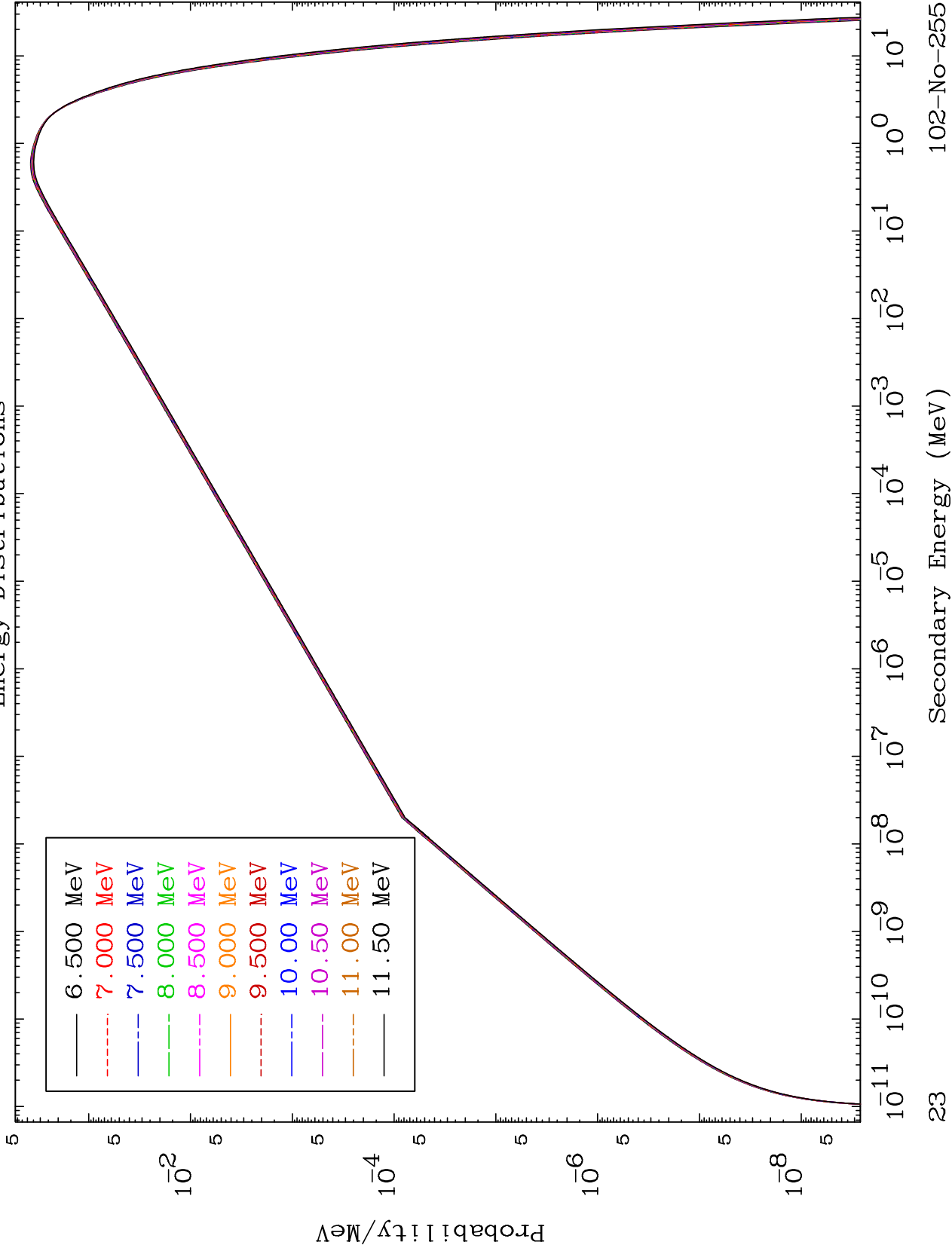
Fission Energy Distributions



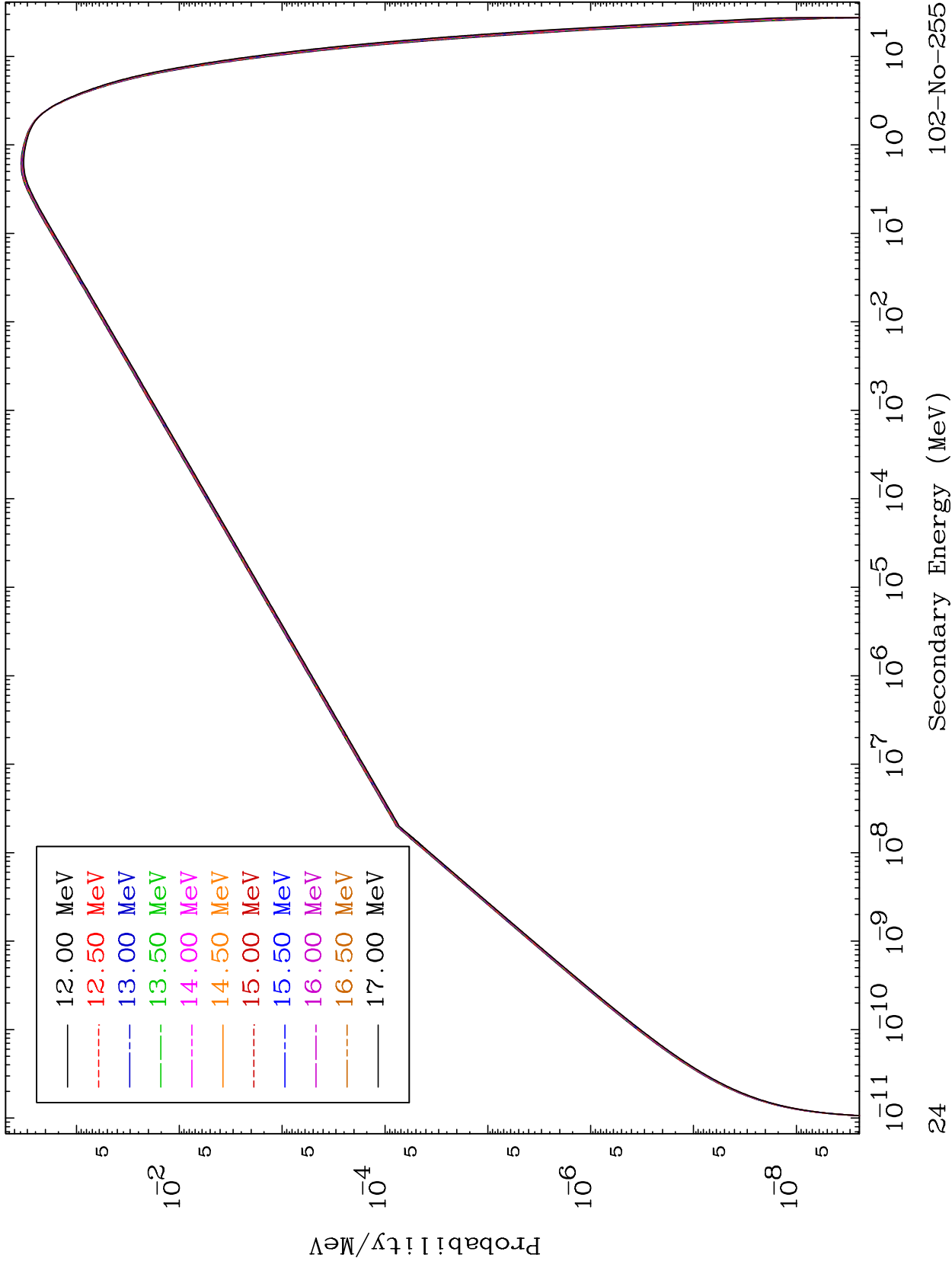
MAT 9973

Fission Energy Distributions

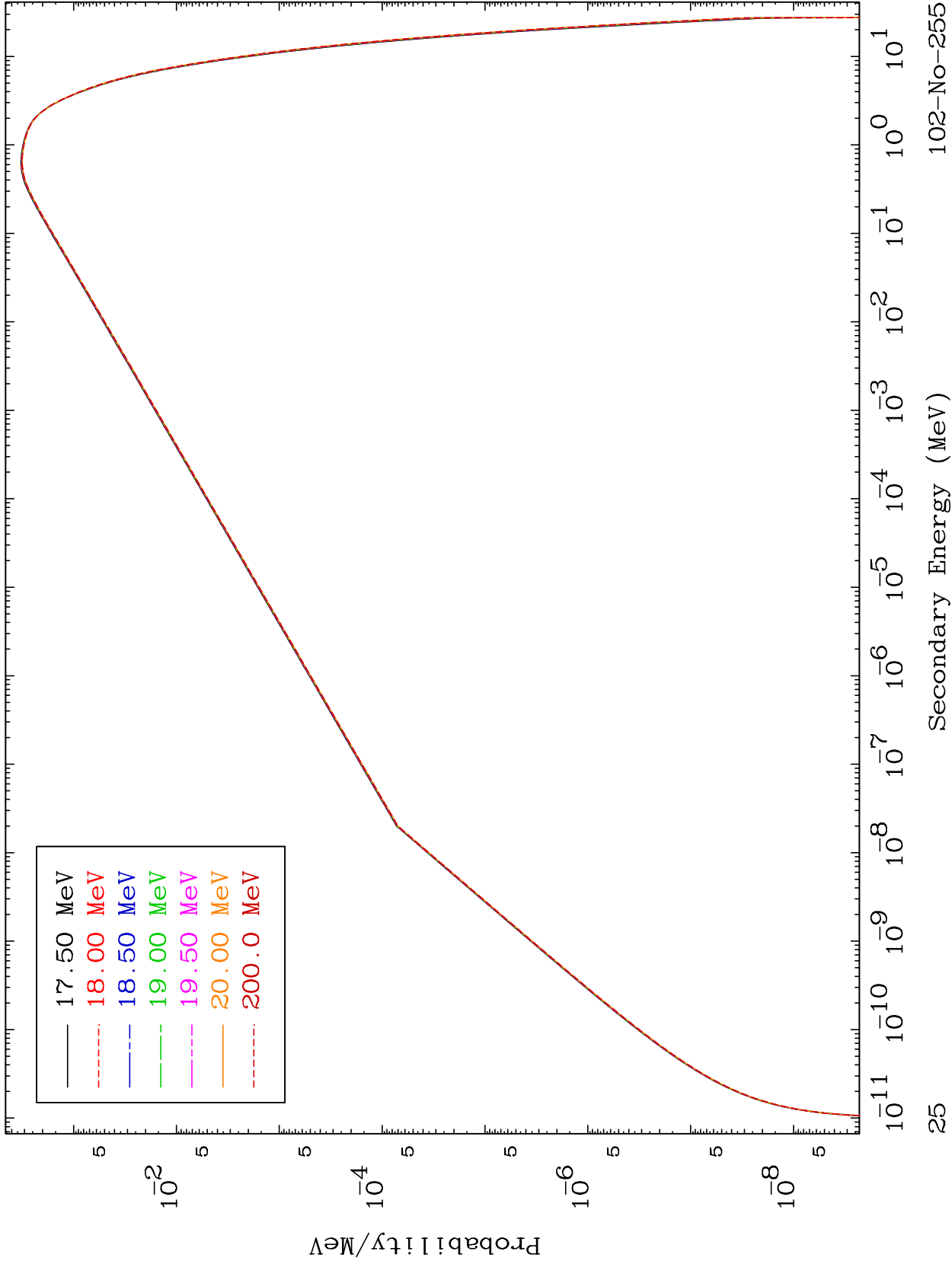
102-No-255



Fission Energy Distributions



Fission Energy Distributions

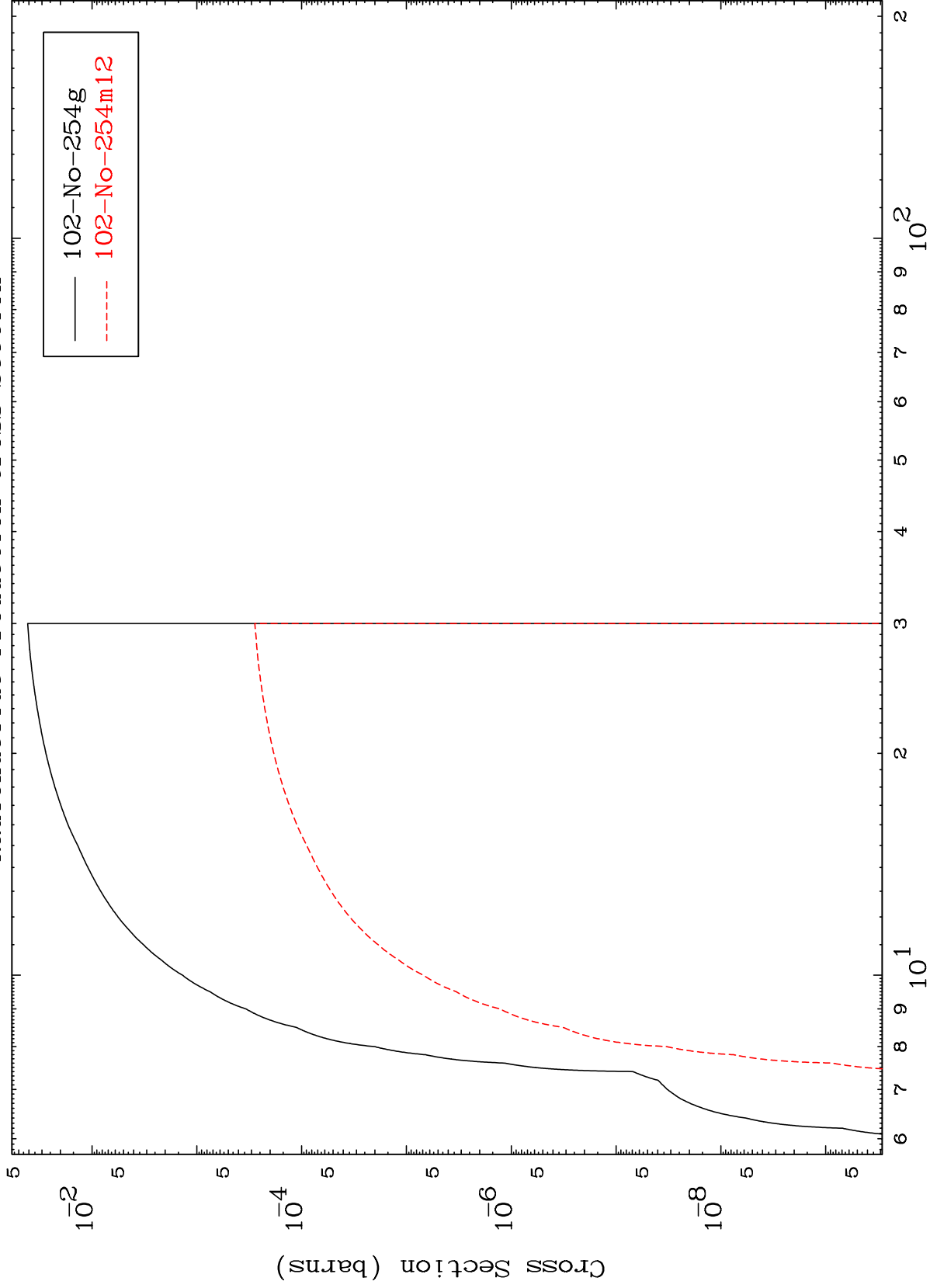


MAT 9973

(n,2n)

102-No-255

Radionuclide Production Cross Section



26

Incident Energy (MeV)

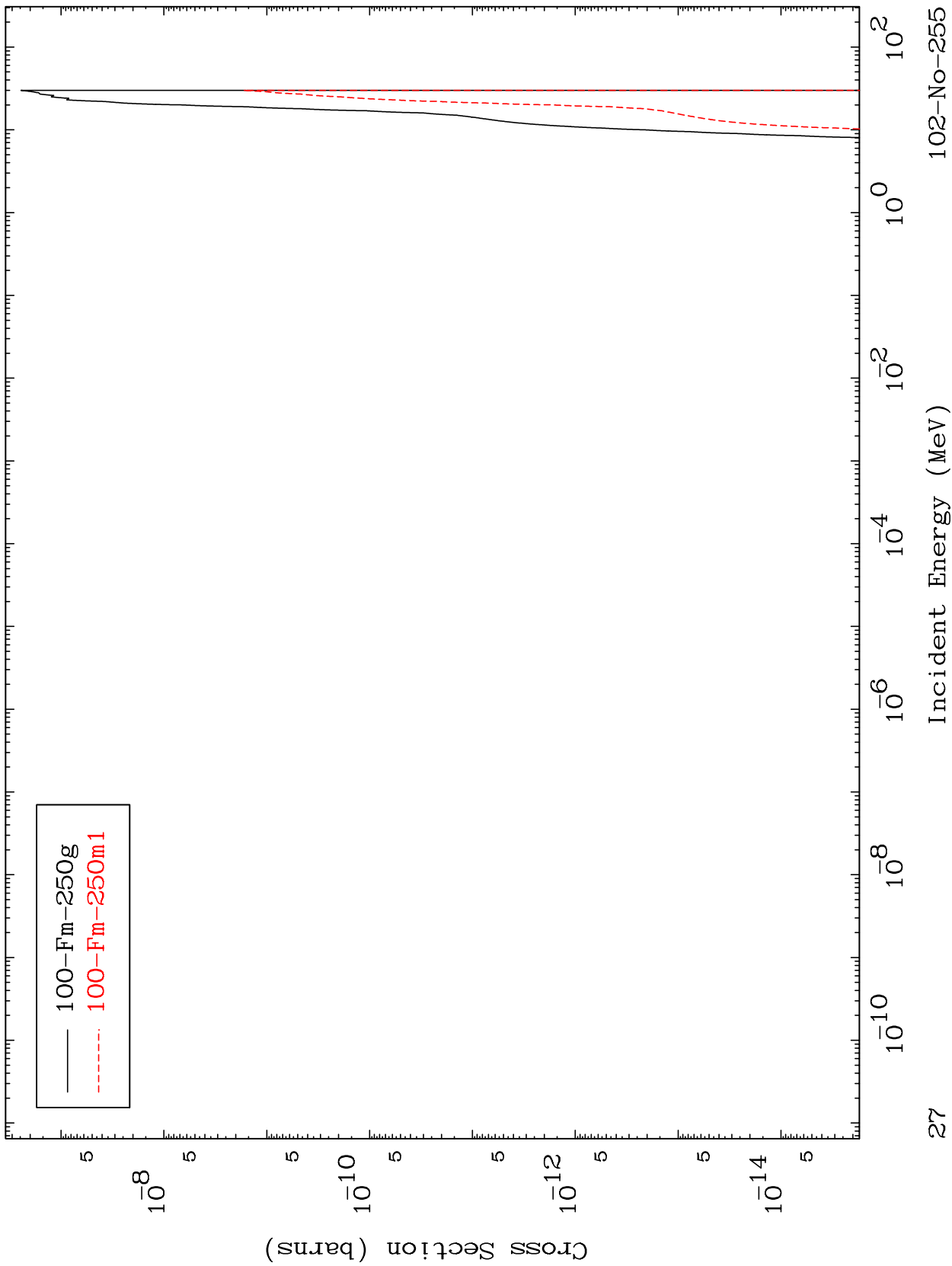
102-No-255

MAT 9973

(n,2n) α

102-No-255

Radionuclide Production Cross Section

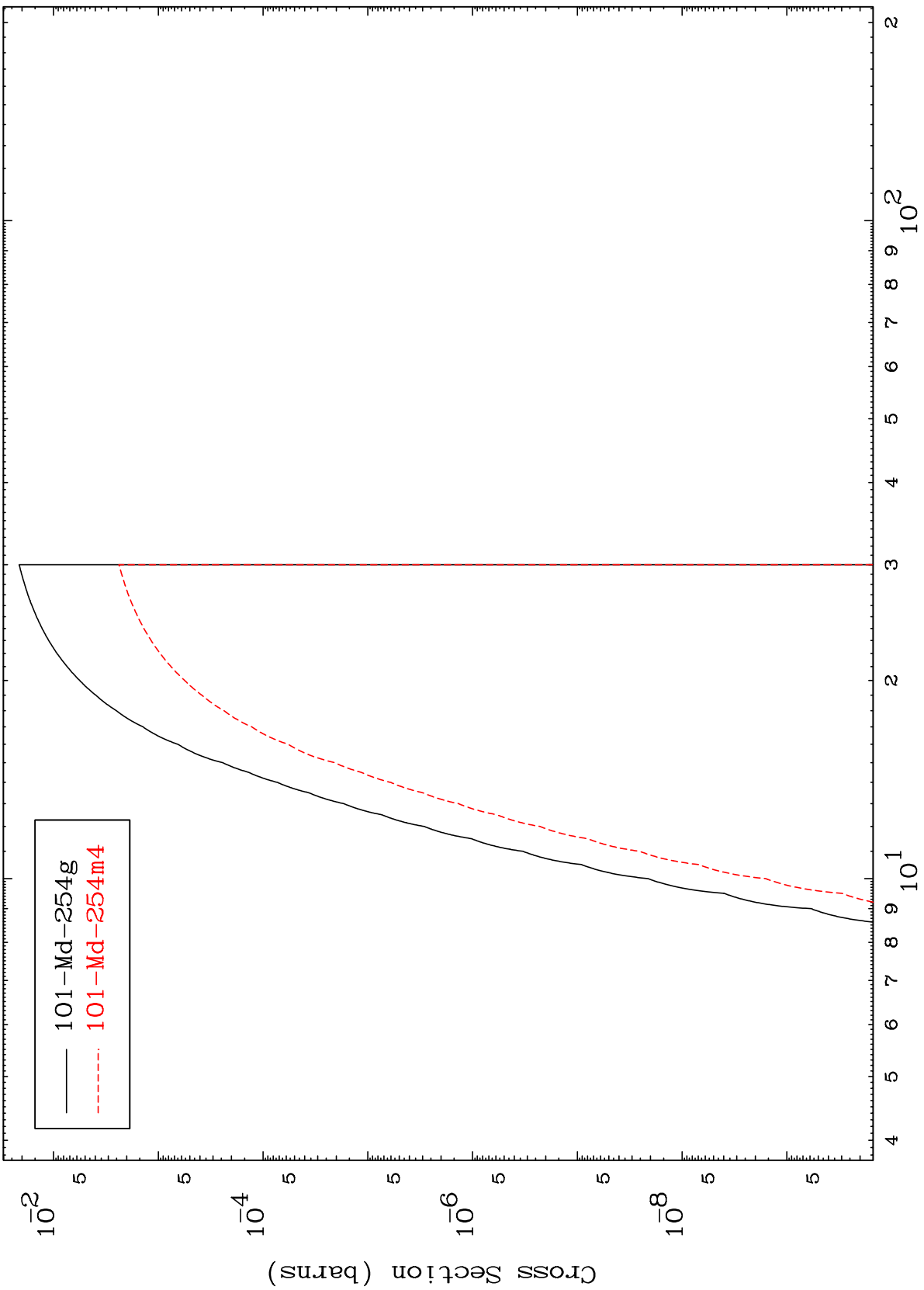


MAT 9973

(n,n') p

102-No-255

Radionuclide Production Cross Section



28

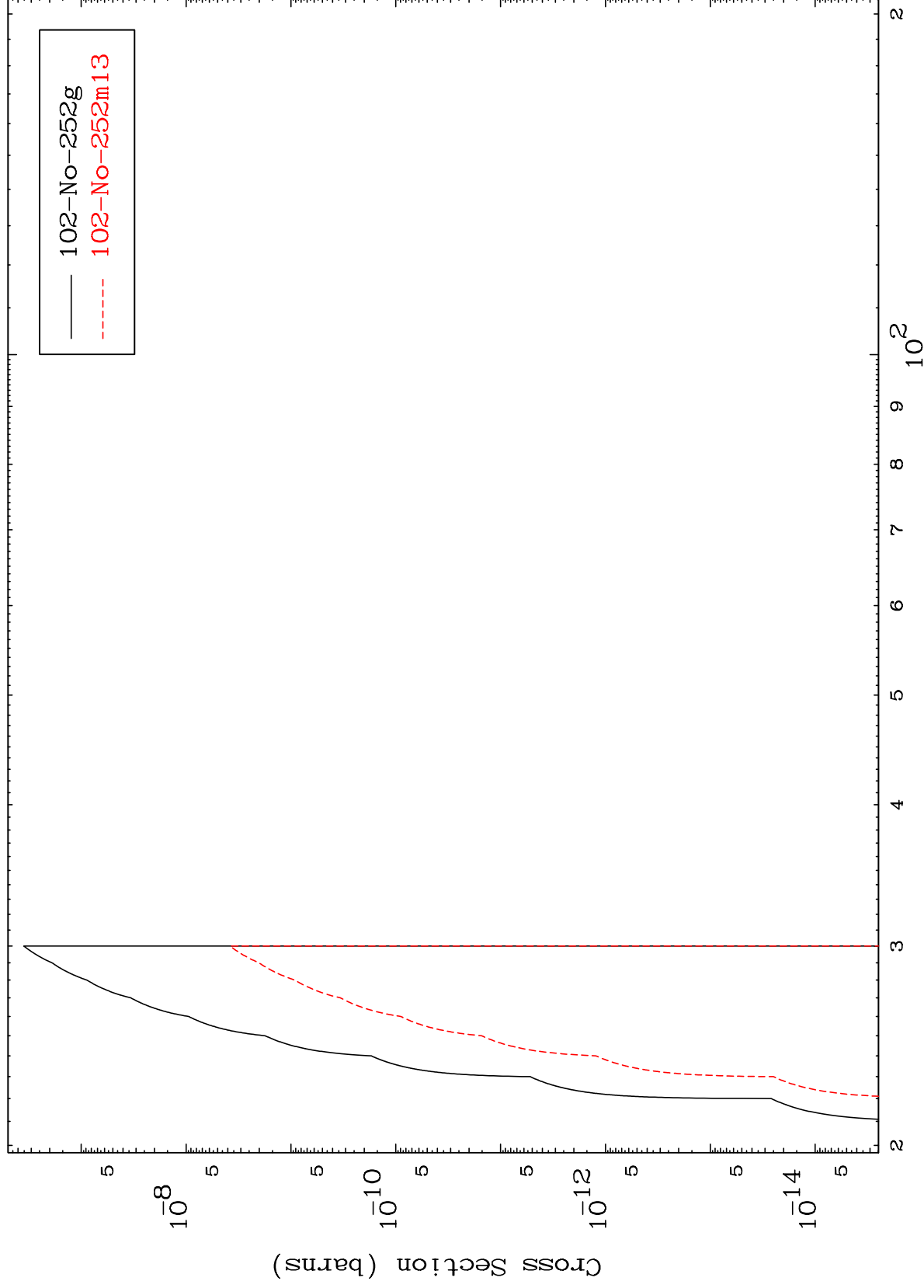
Incident Energy (MeV)

102-No-255

MAT 9973

102-No-255

(n,4n)
Radionuclide Production Cross Section



102-No-255

Incident Energy (MeV)

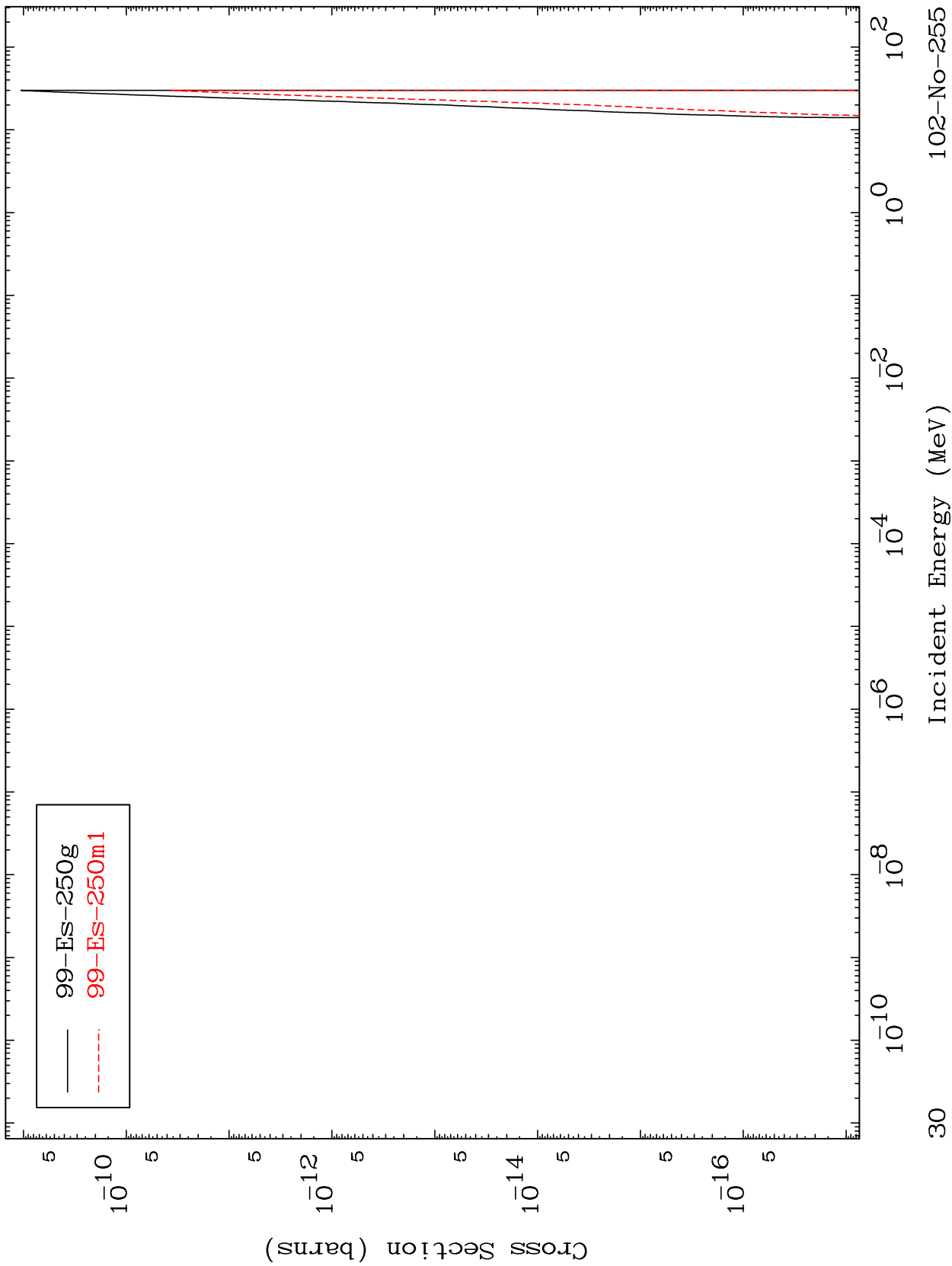
29

MAT 9973

(n,n') p α

102-No-255

Radionuclide Production Cross Section



MAT 9973

(n,d)

102-No-255

