

Program EVALPLOT  
(Version 2021-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

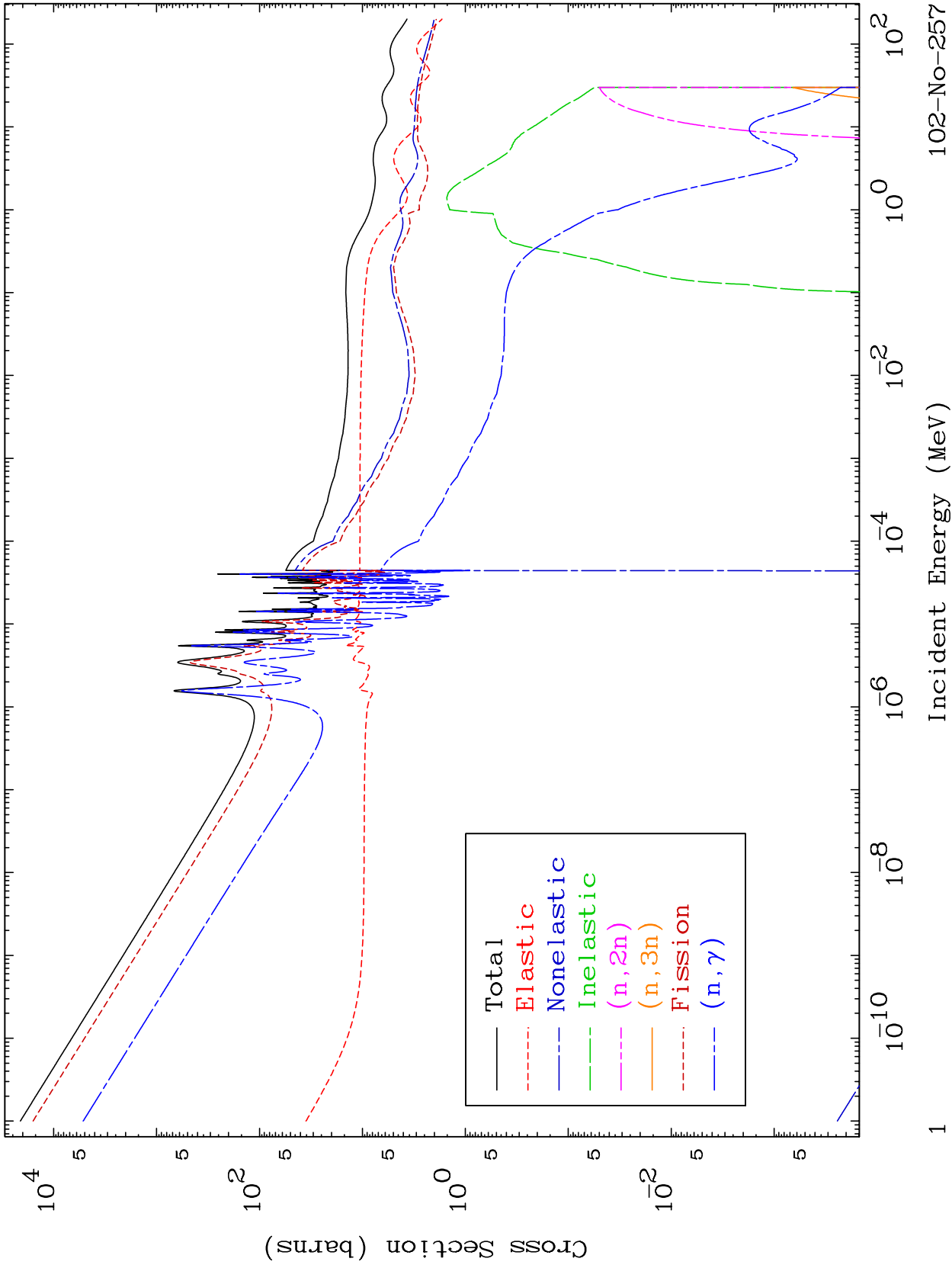
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9975

Neutron Major  
293 Kelvin Cross Sections

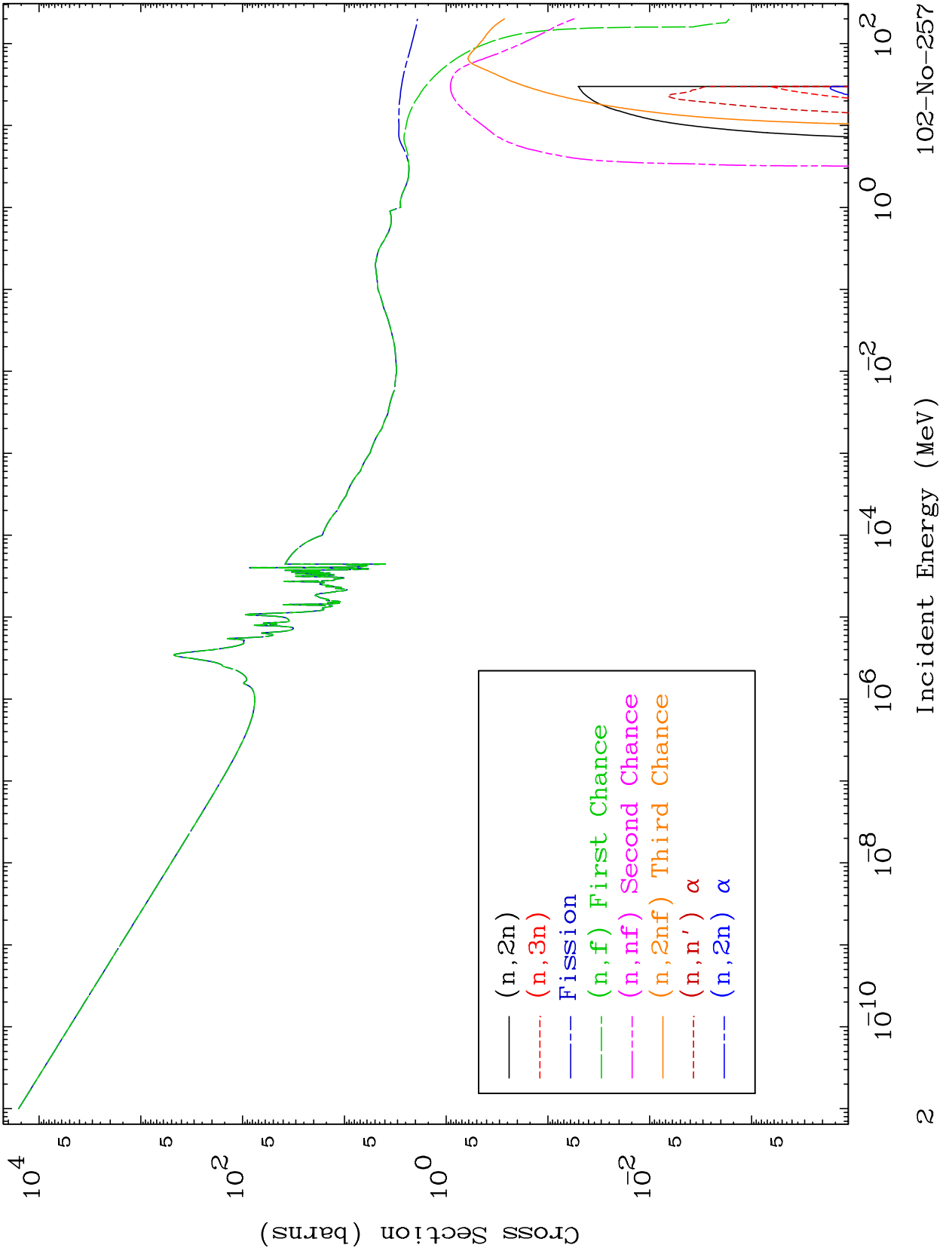
102-No-257



MAT 9975

Neutron Absorption  
293 Kelvin Cross Sections

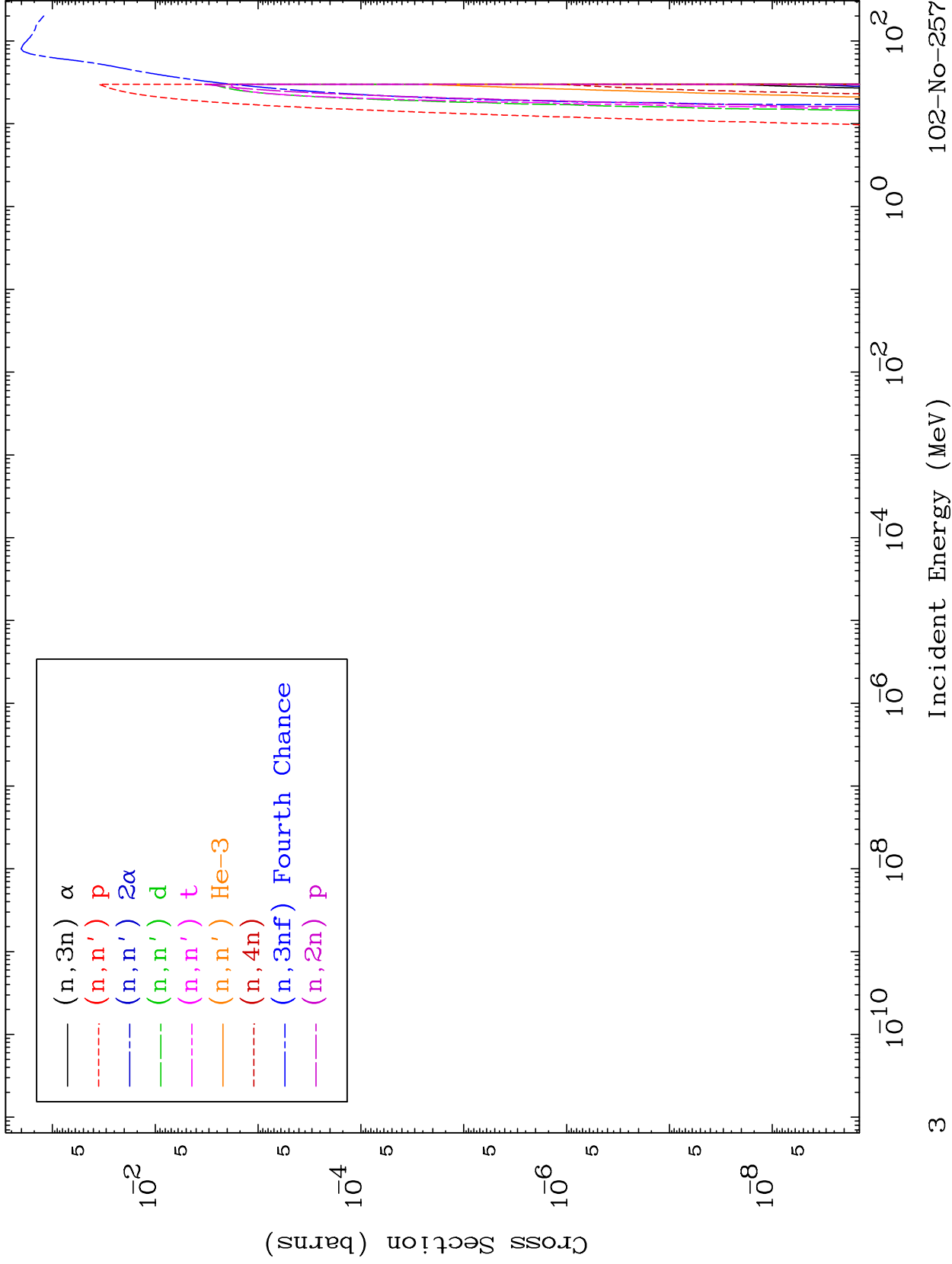
102-No-257



MAT 9975

Neutron Absorption  
293 Kelvin Cross Sections

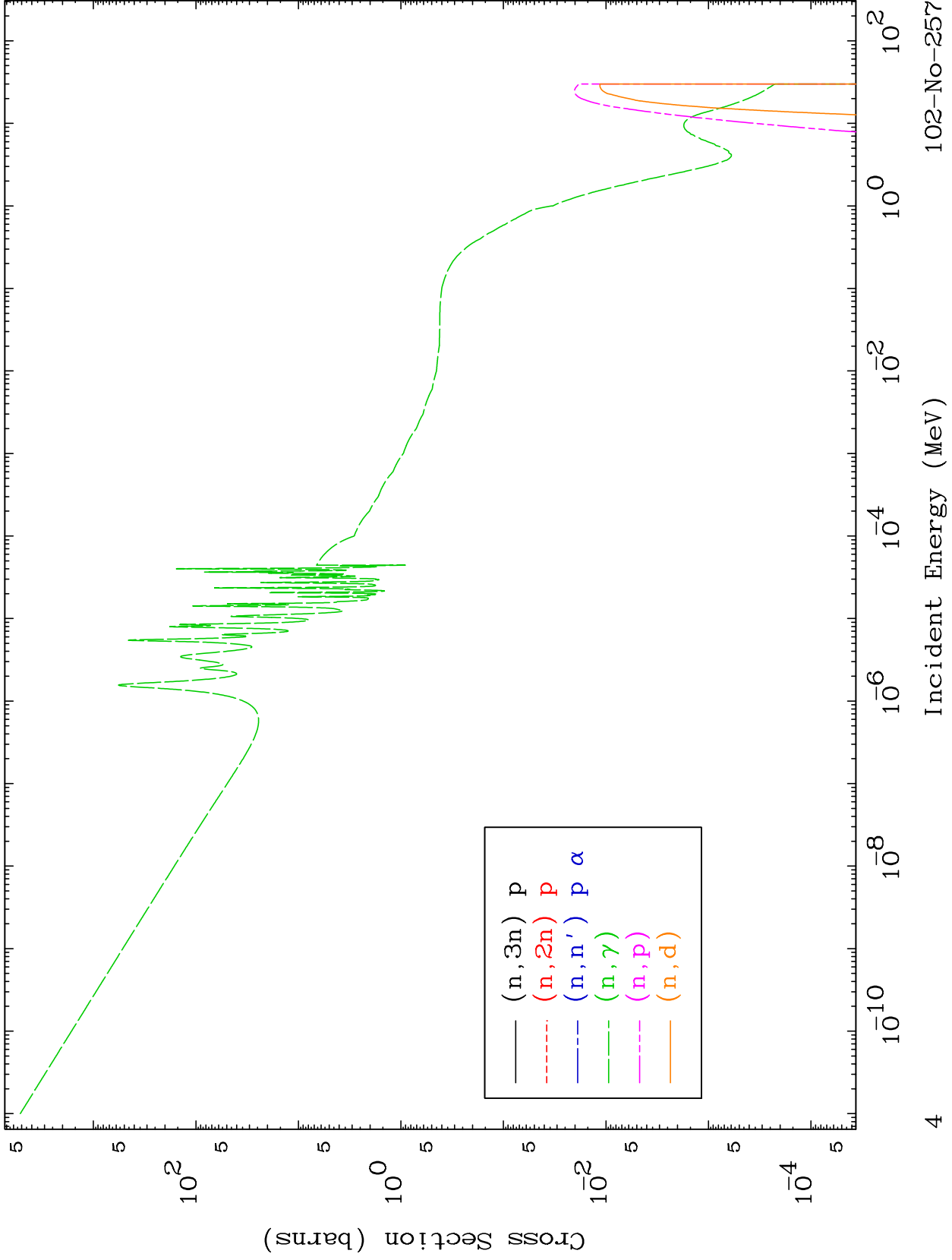
102-No-257



MAT 9975

Neutron Absorption  
293 Kelvin Cross Sections

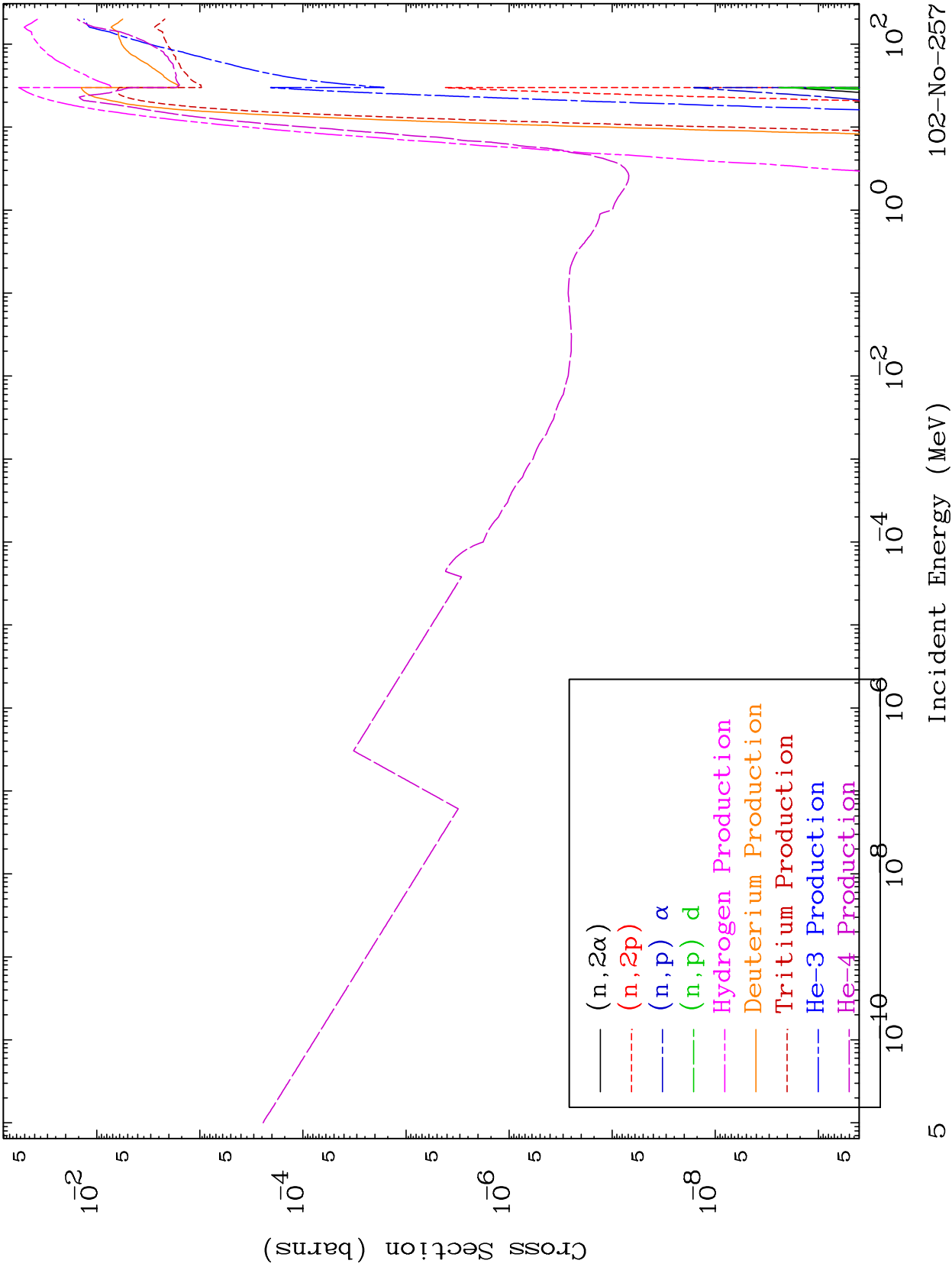
102-No-257



MAT 9975

Neutron Absorption  
293 Kelvin Cross Sections

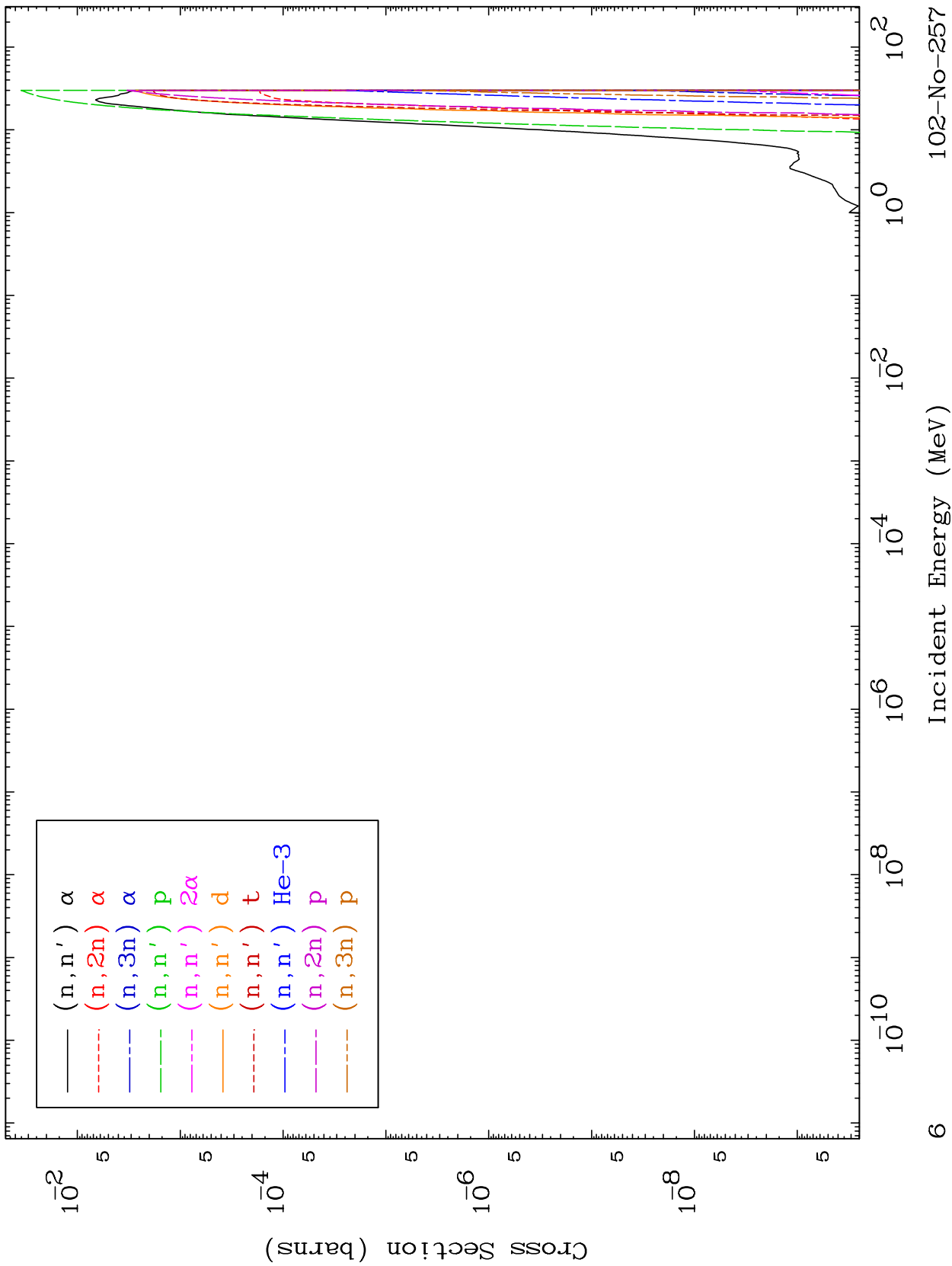
102-No-257



MAT 9975

Charged Particle  
293 Kelvin Cross Sections

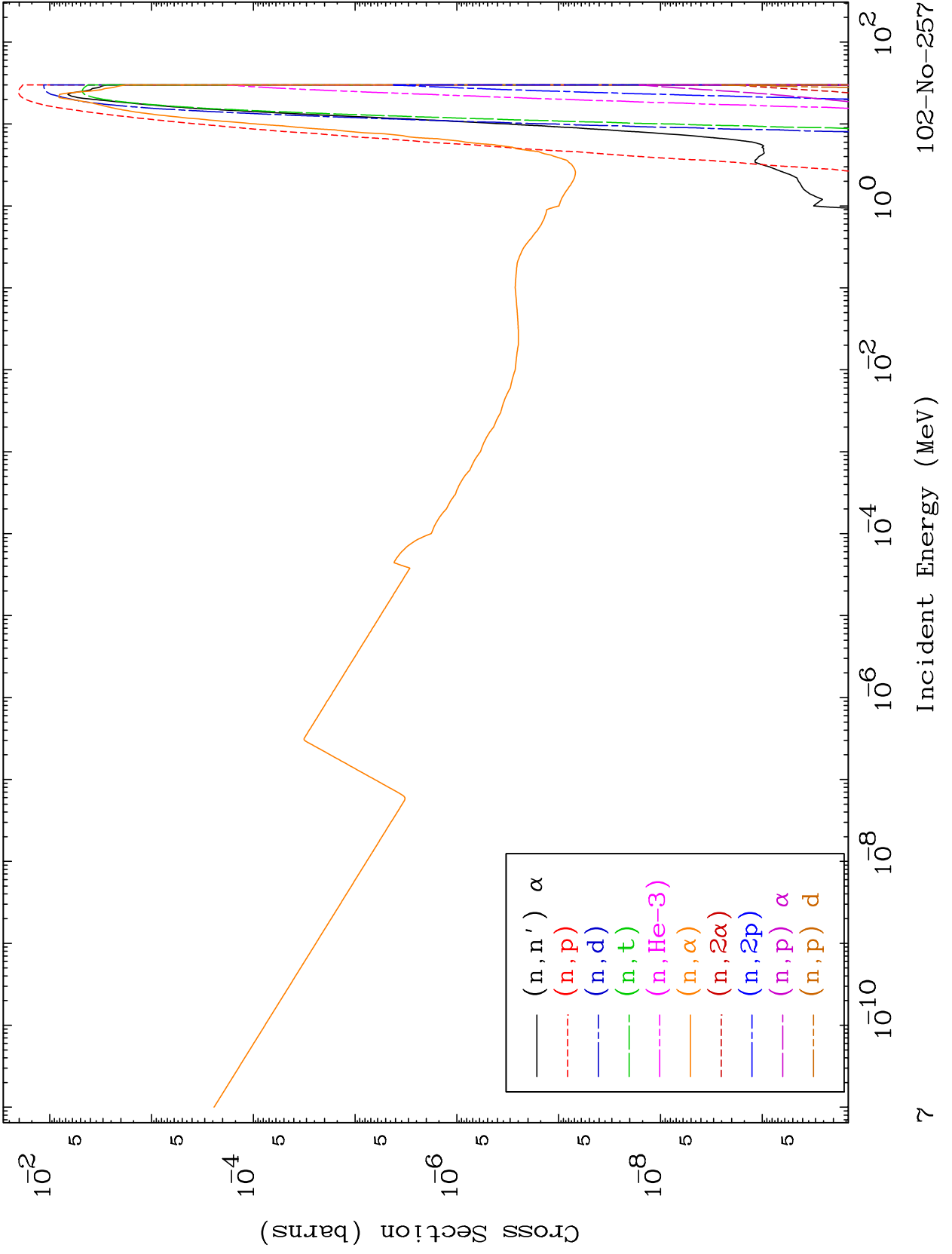
102-No-257



MAT 9975

Charged Particle  
293 Kelvin Cross Sections

102-No-257

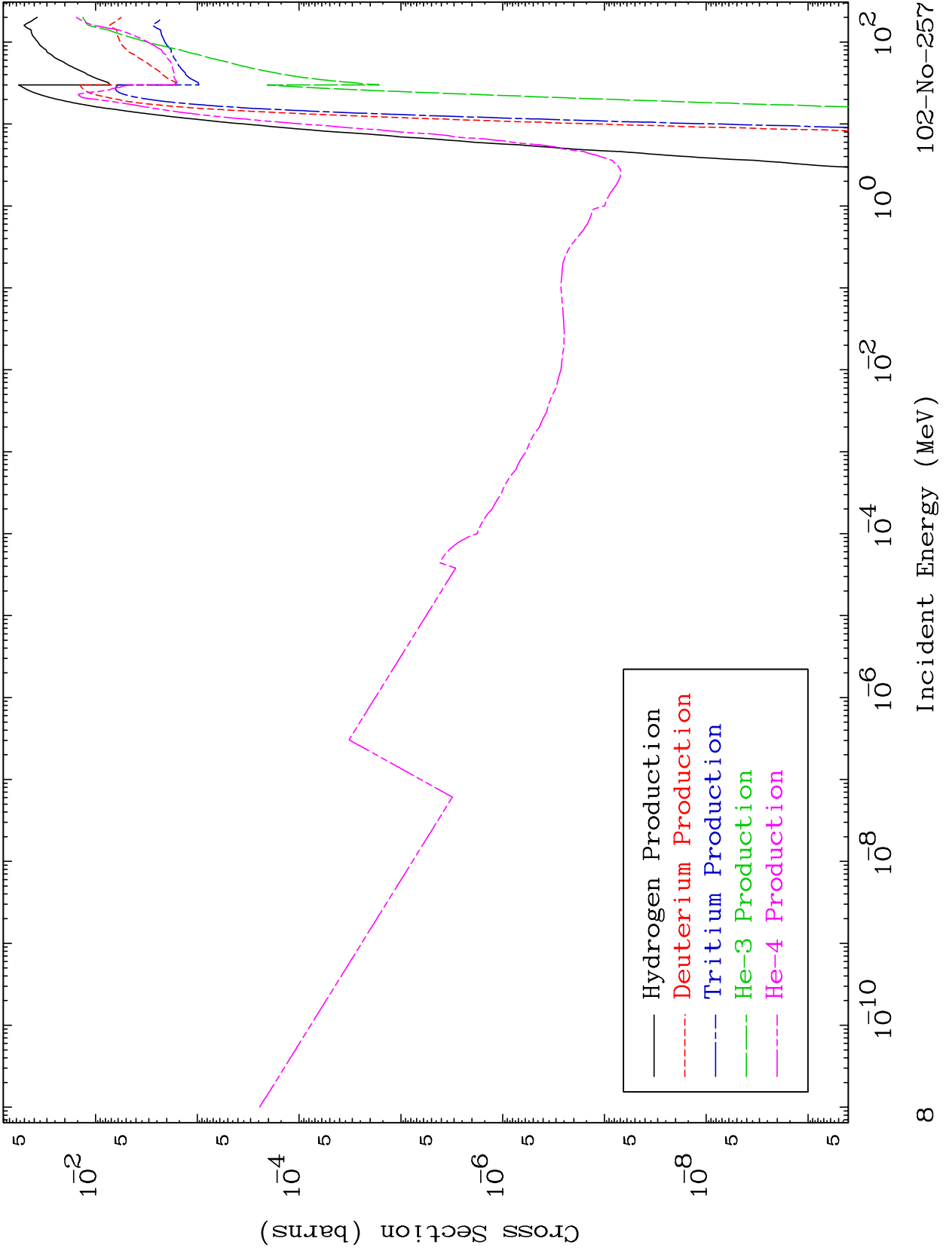




MAT 9975

Particle Production  
293 Kelvin Cross Sections

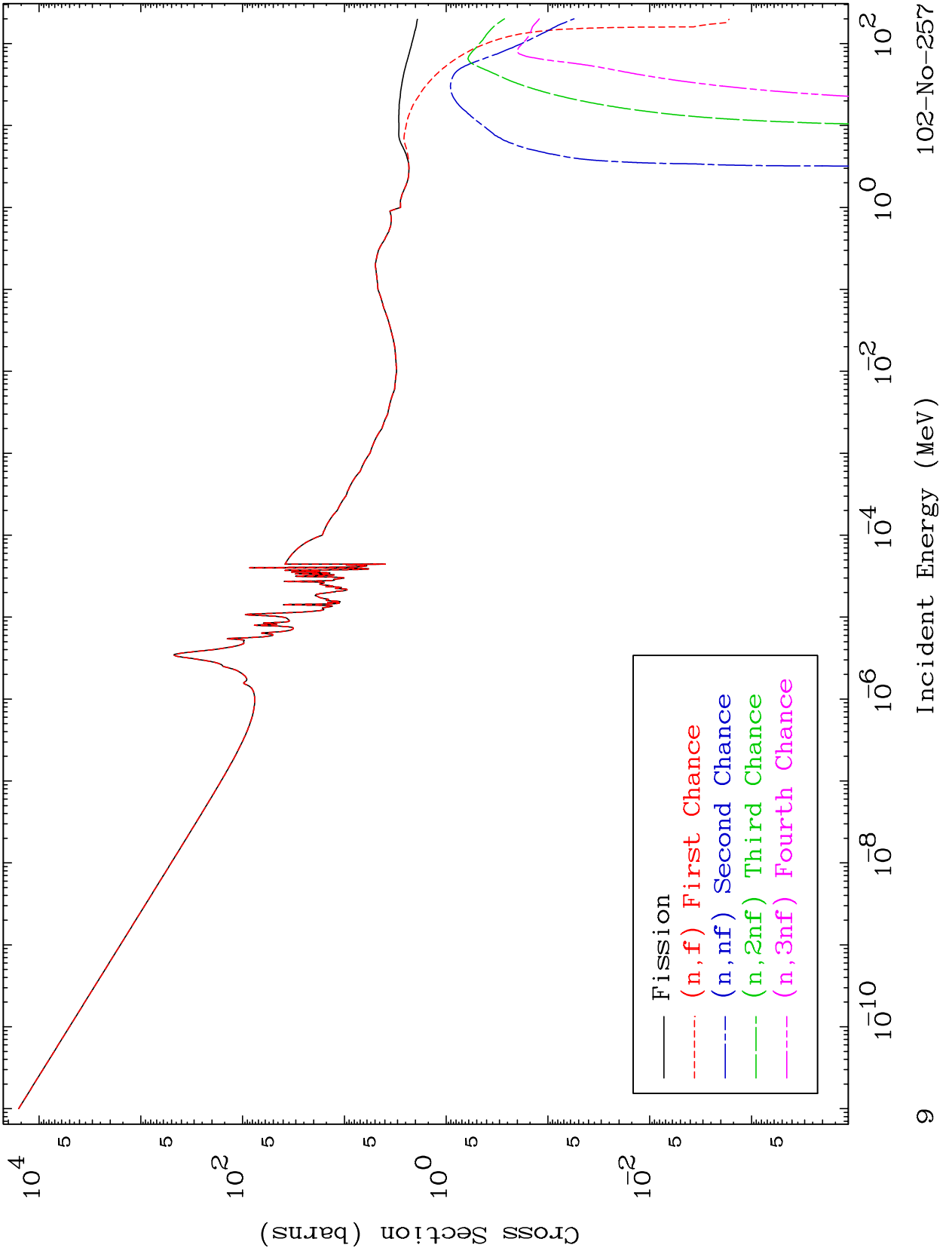
102-No-257



MAT 9975

Fission  
293 Kelvin Cross Sections

102-No-257

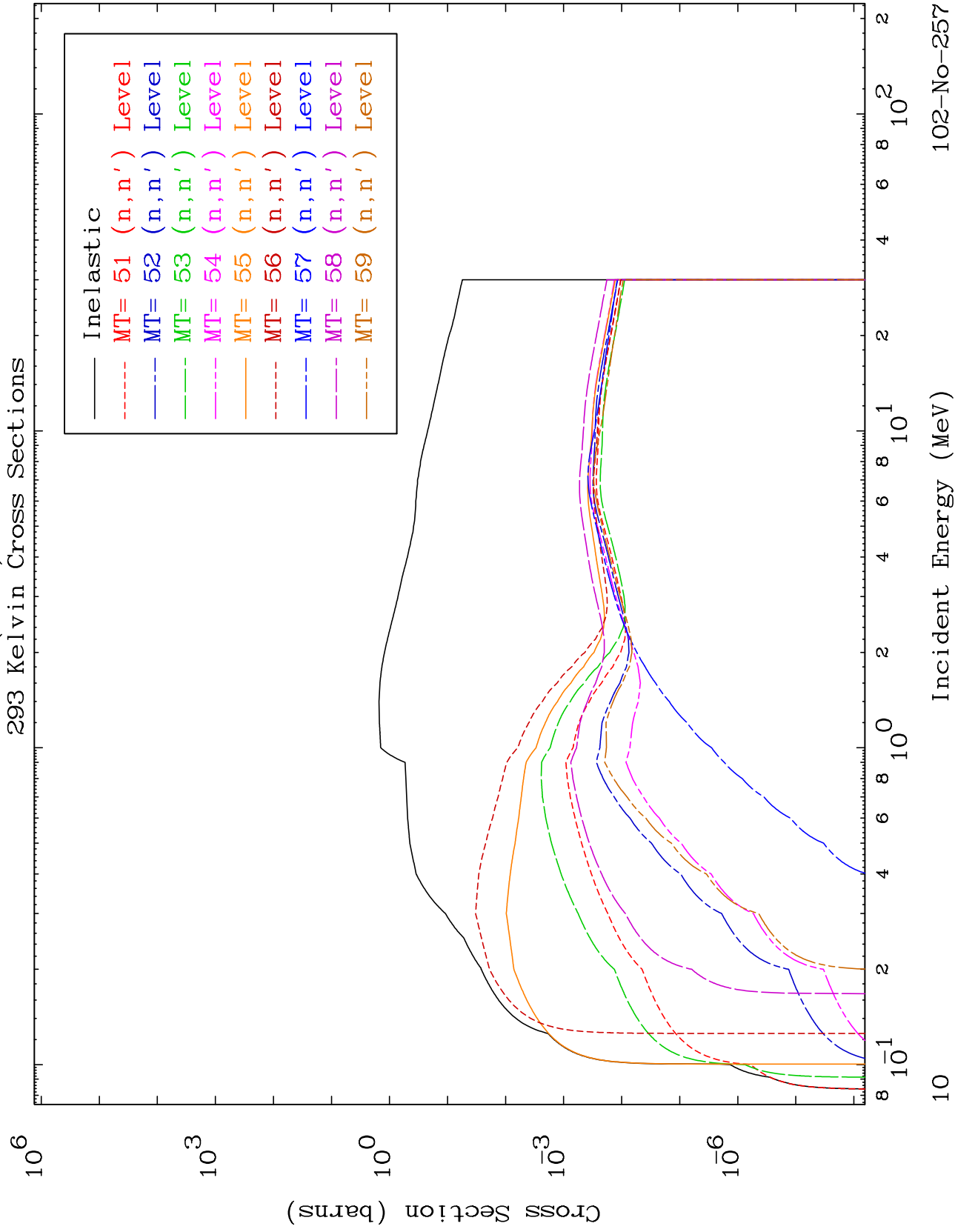


MAT 9975

(n,n') Levels

293 Kelvin Cross Sections

102-No-257



Incident Energy (MeV)

102-No-257

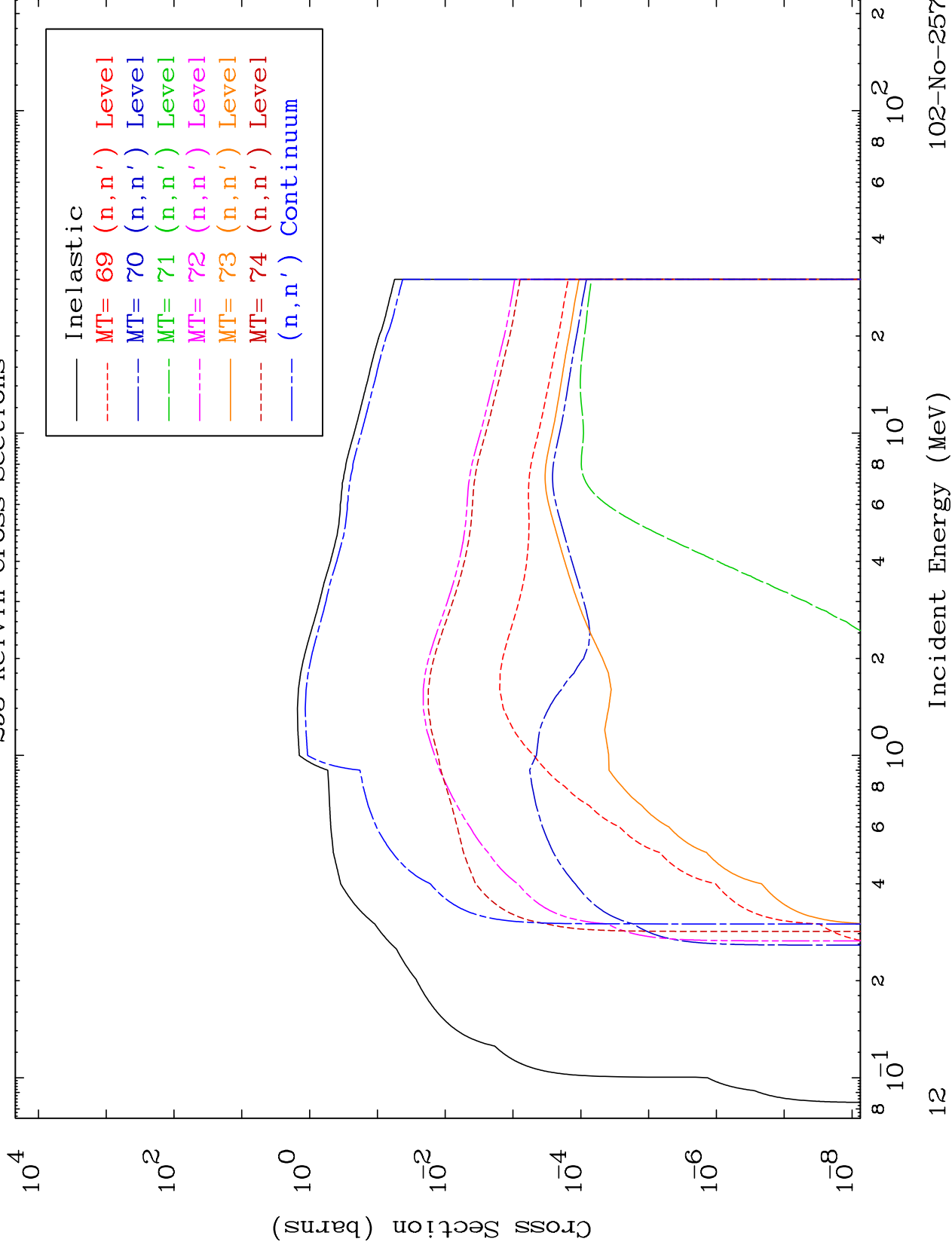


MAT 9975

(n,n') Levels

102-No-257

293 Kelvin Cross Sections



12

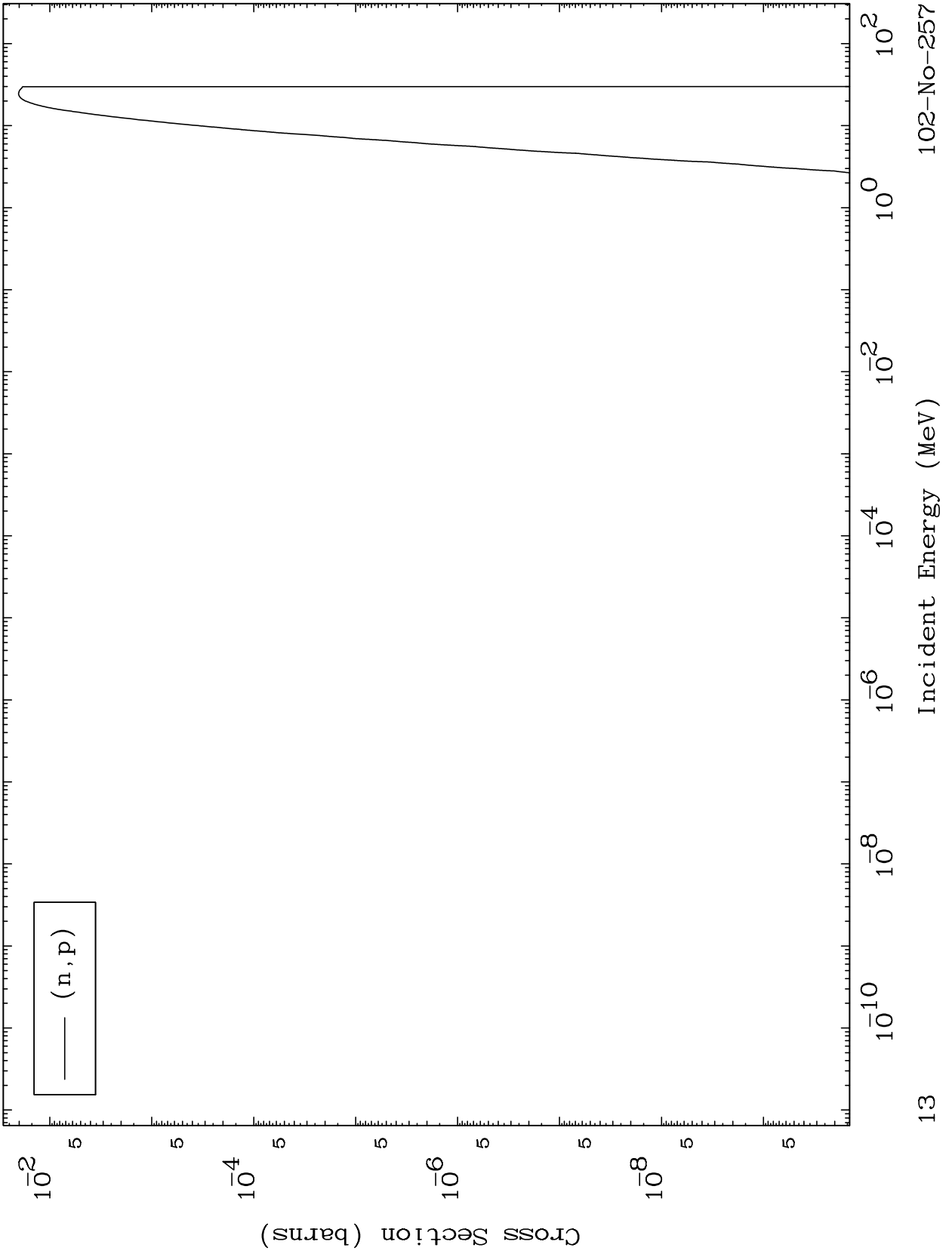
Incident Energy (MeV)

102-No-257

MAT 9975

(n,p) Levels  
293 Kelvin Cross Sections

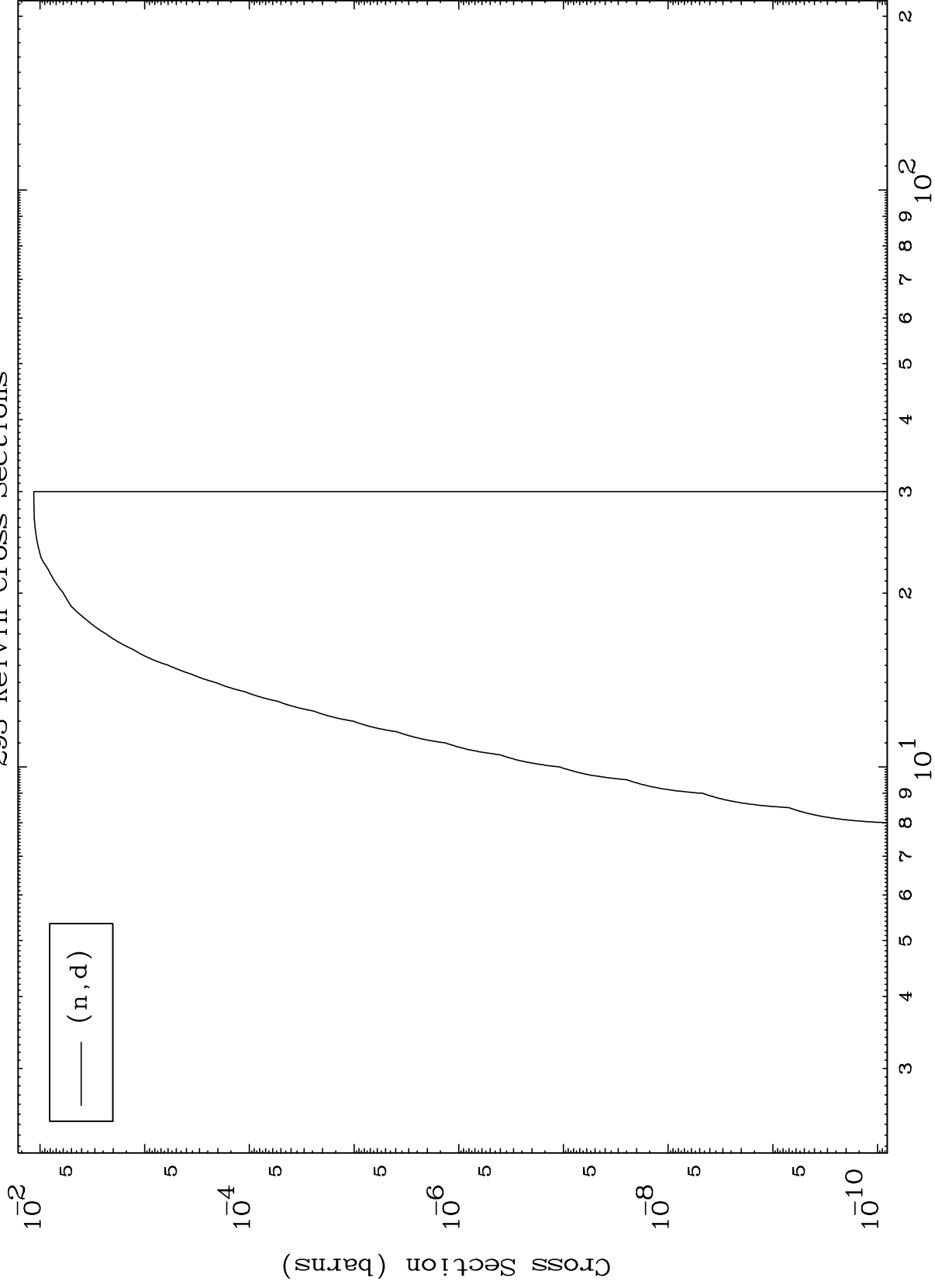
102-No-257



MAT 9975

(n,d) Levels  
293 Kelvin Cross Sections

102-No-257



14

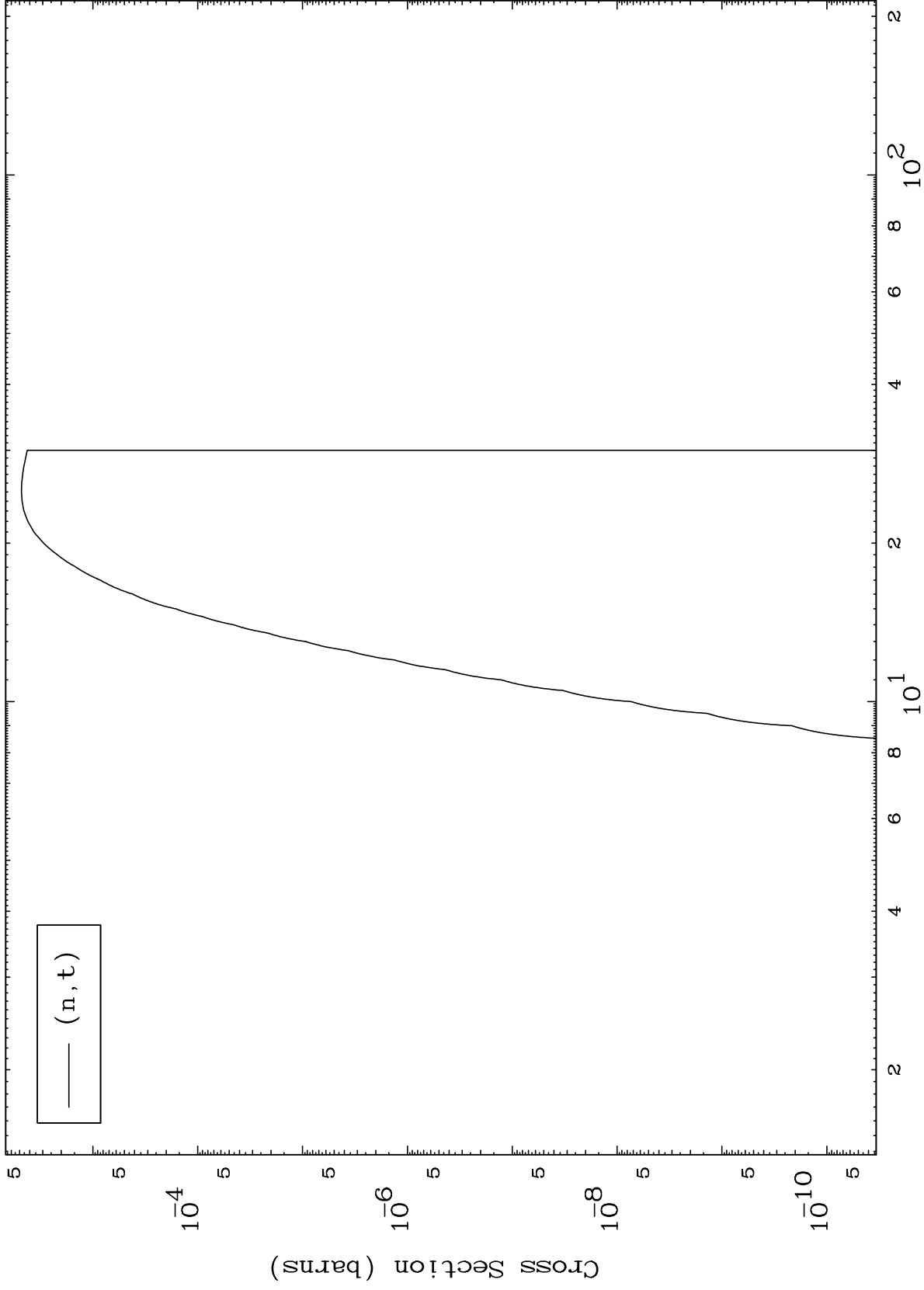
Incident Energy (MeV)

102-No-257

MAT 9975

(n,t) Levels  
293 Kelvin Cross Sections

102-No-257



15

Incident Energy (MeV)

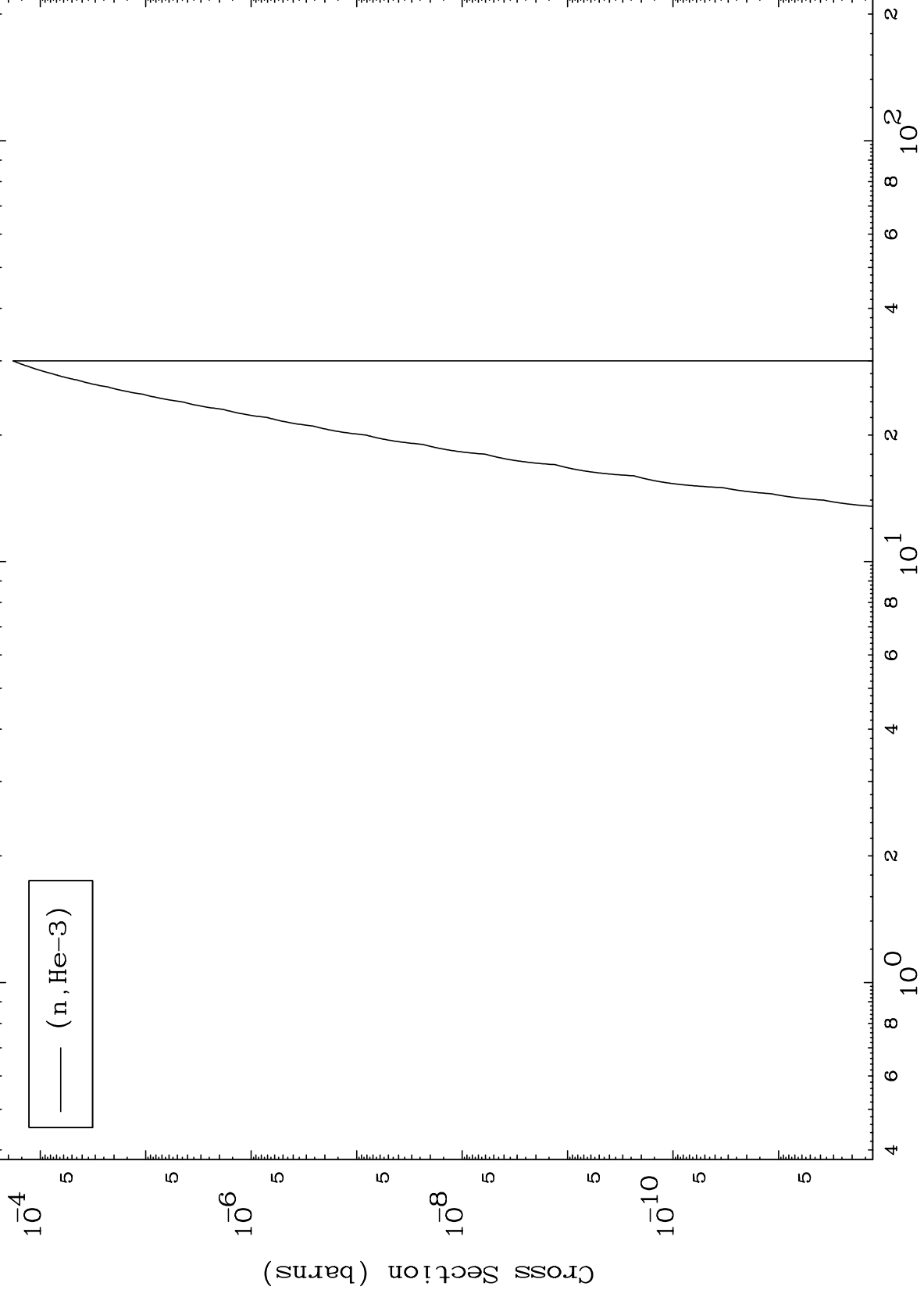
102-No-257



MAT 9975

(n,He3) Levels  
293 Kelvin Cross Sections

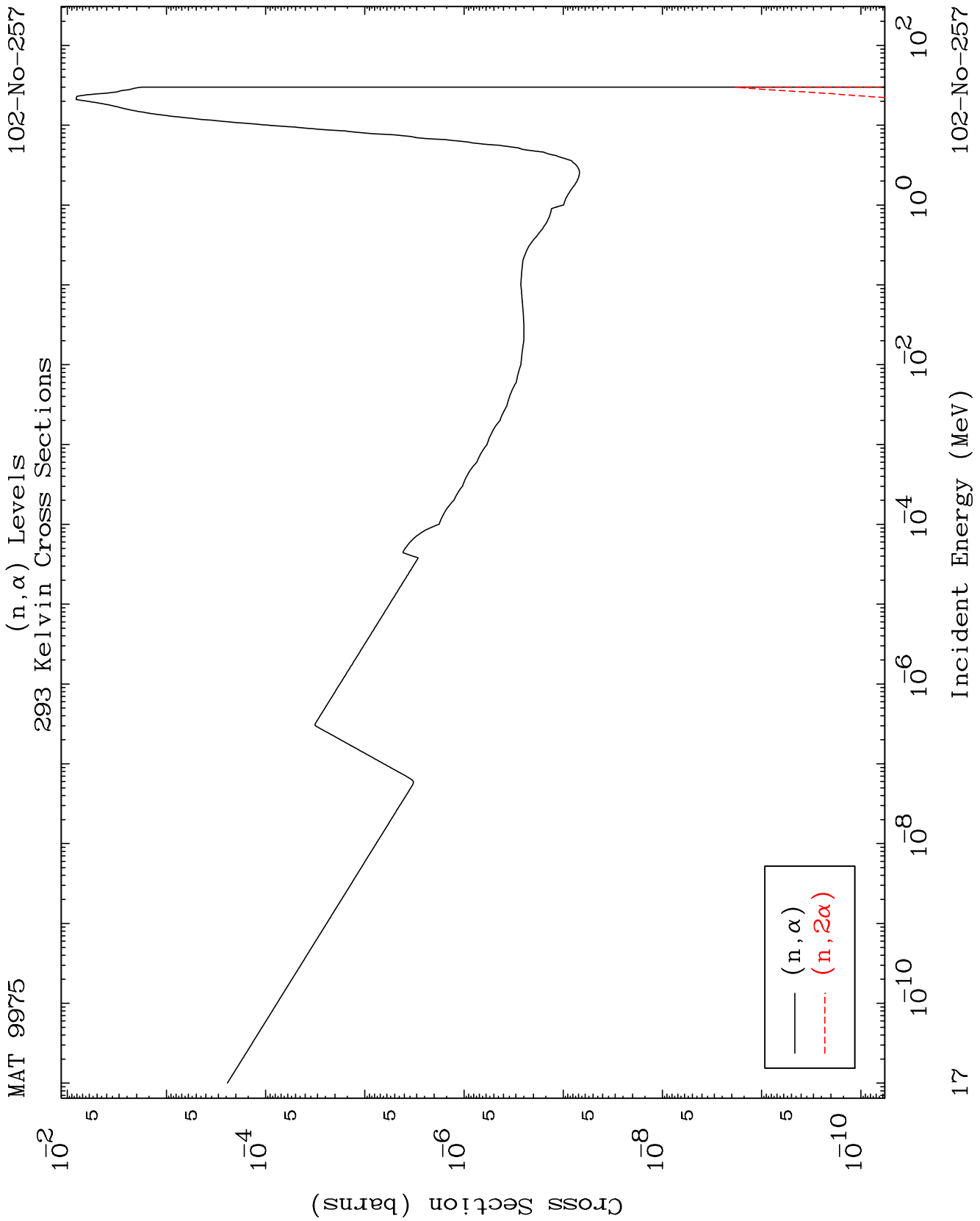
102-No-257



16

Incident Energy (MeV)

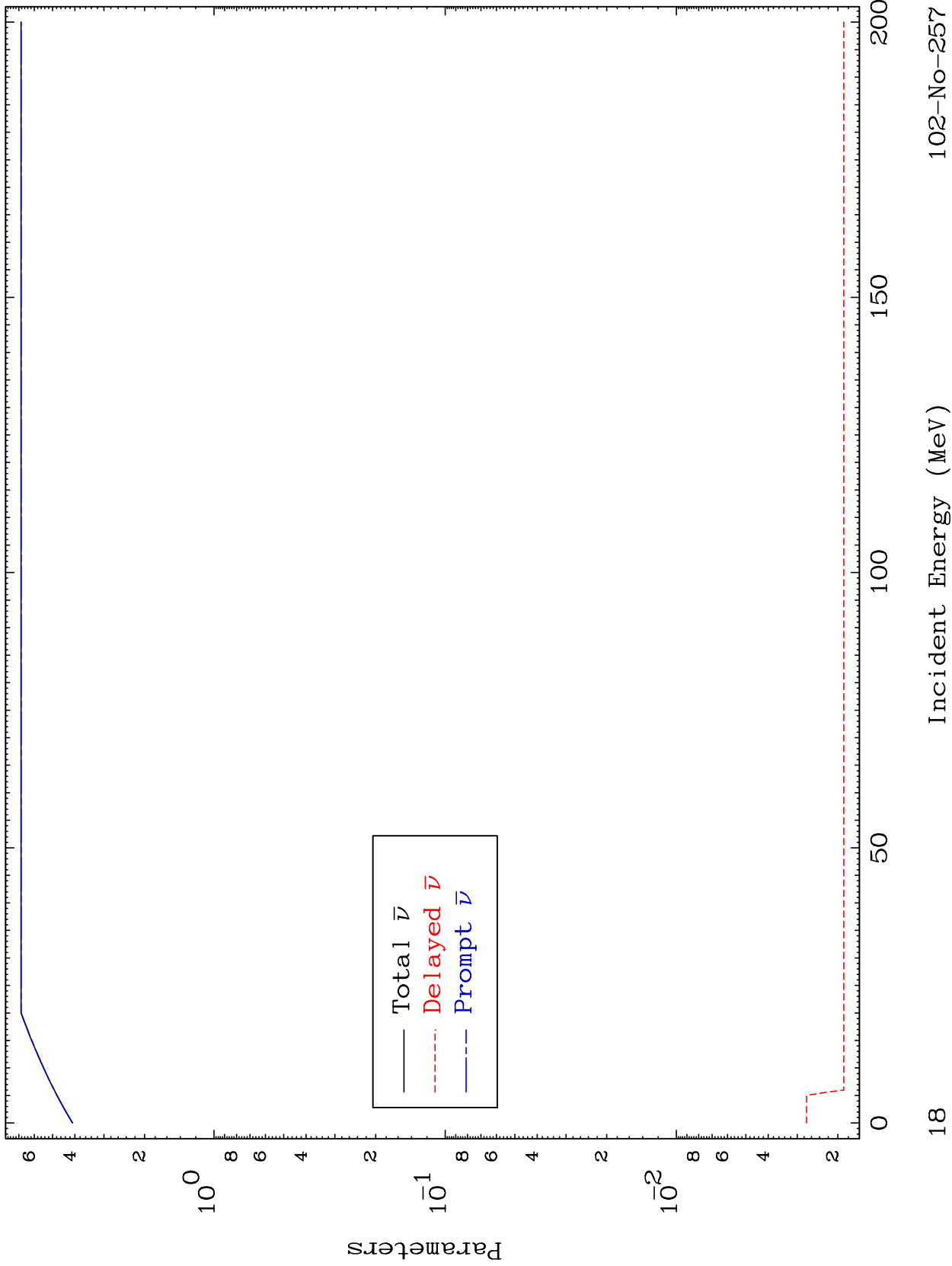
102-No-257



MAT 9975

Energy Release  
Parameters

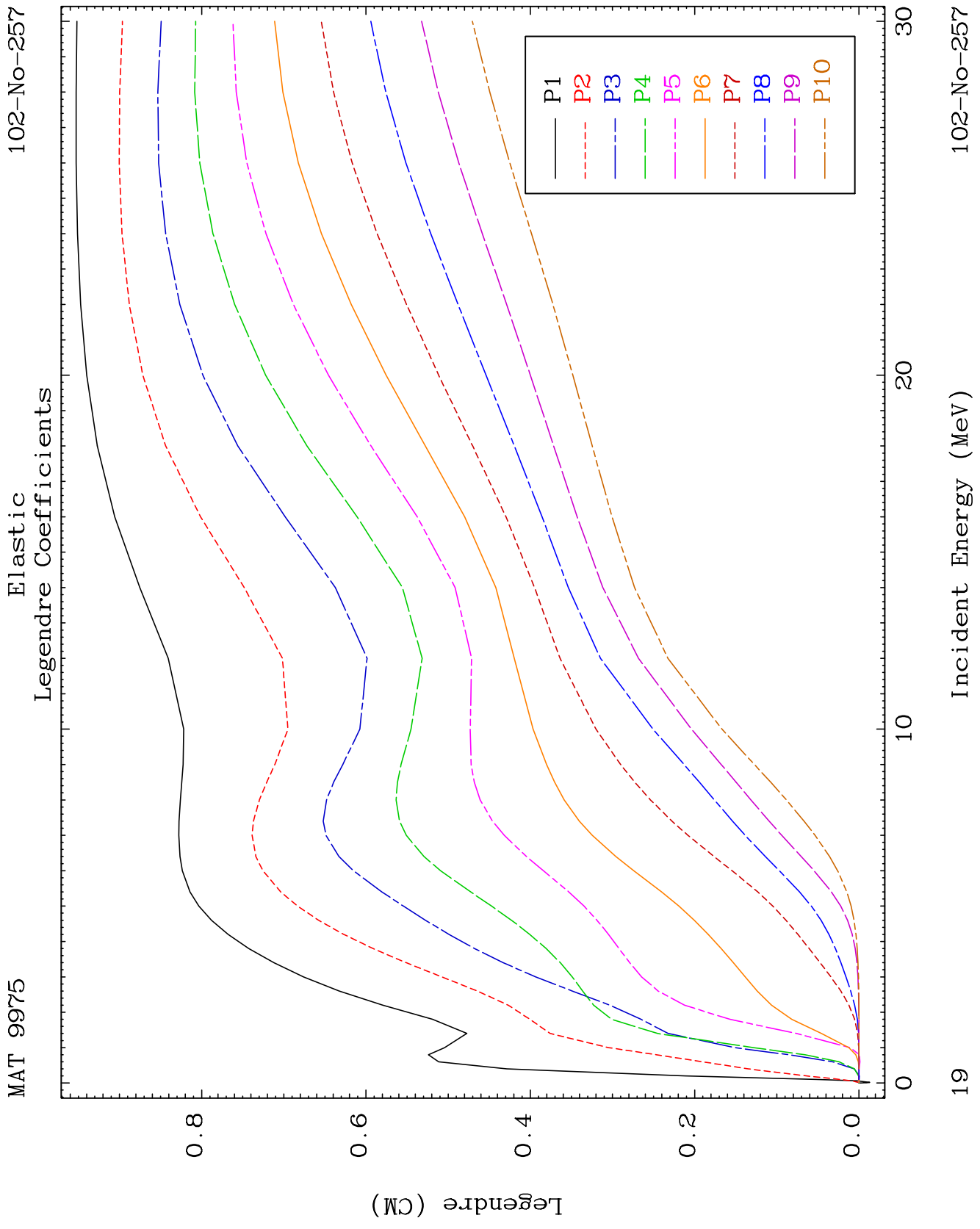
102-No-257

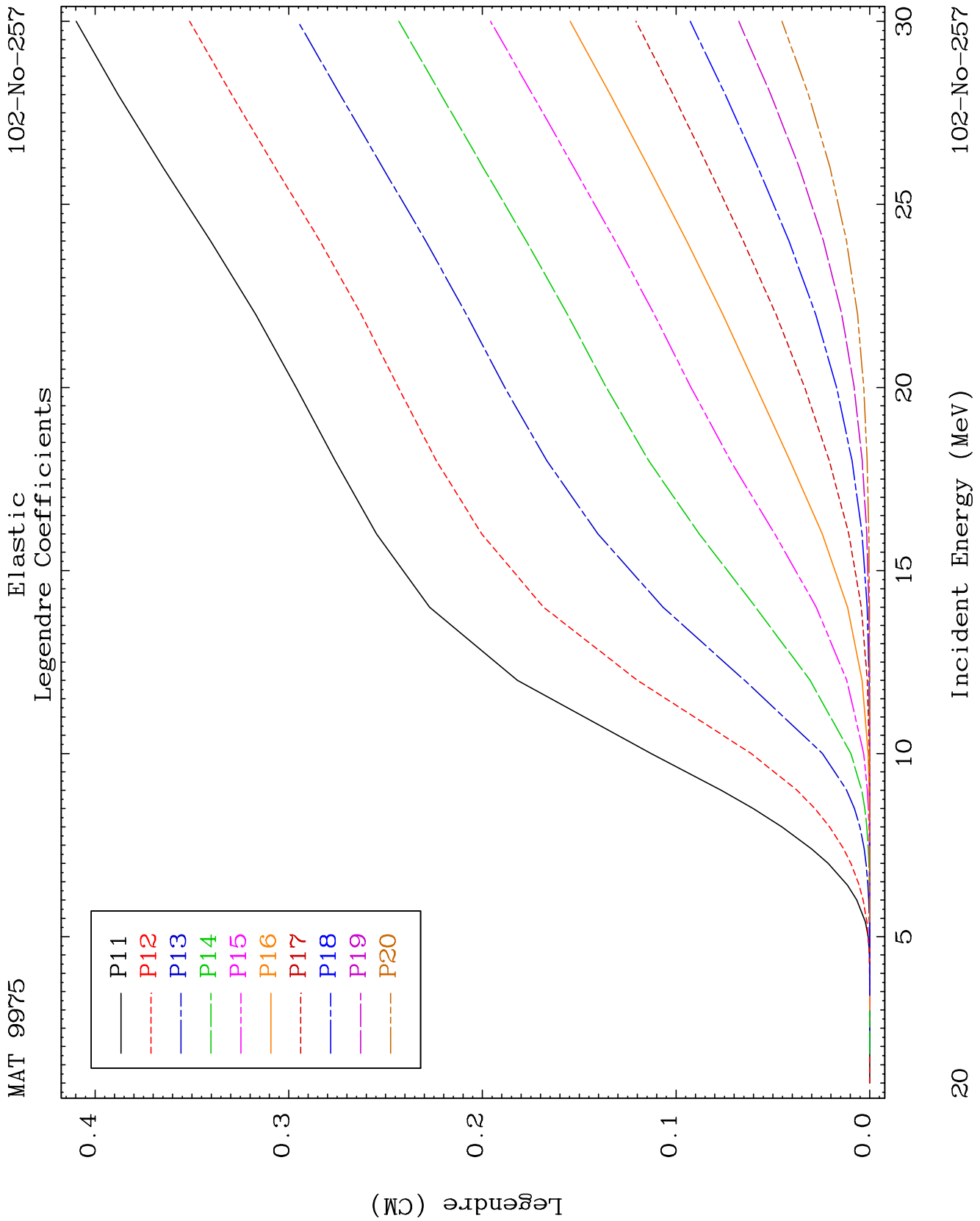


18

Incident Energy (MeV)

102-No-257

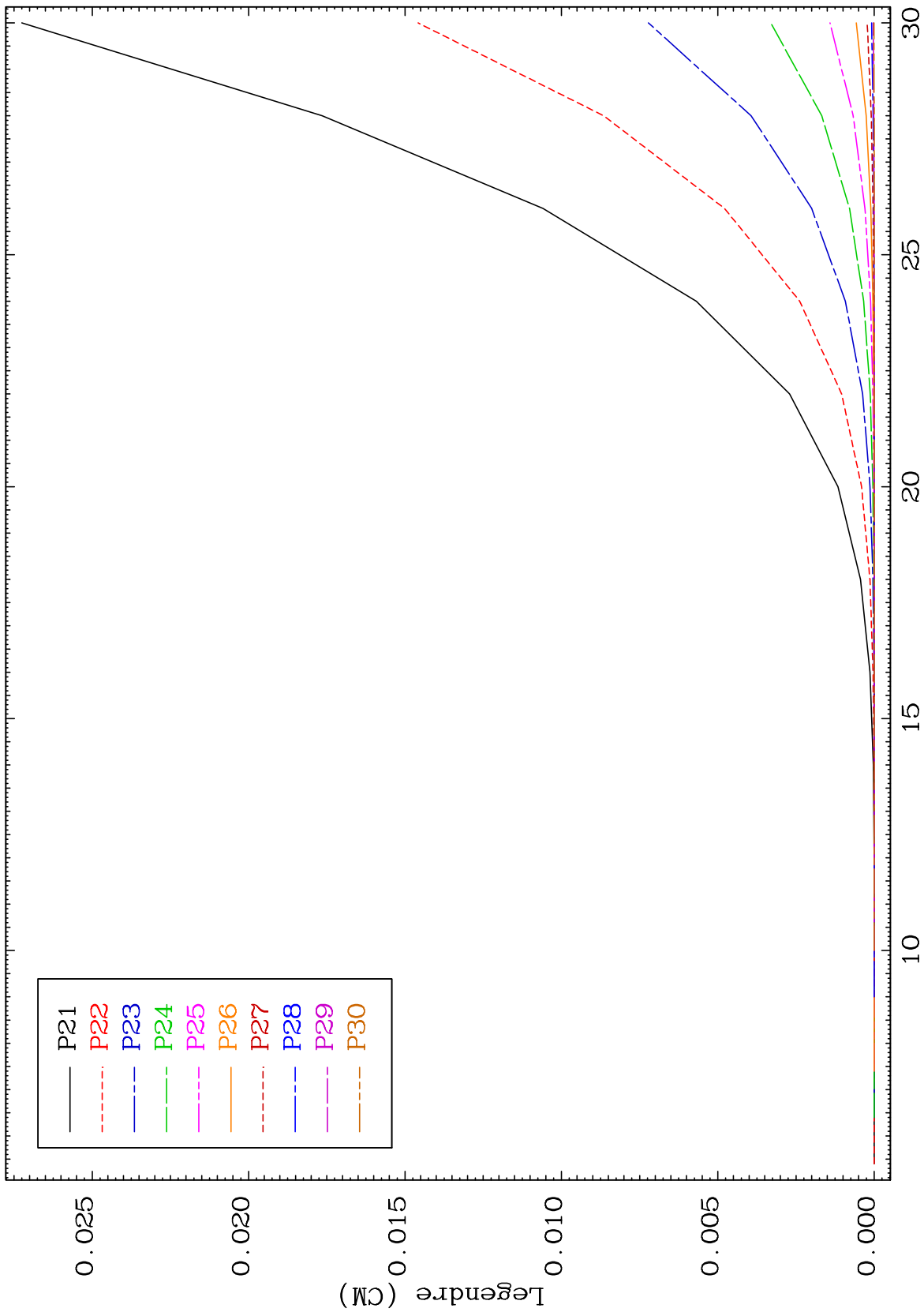




MAT 9975

Elastic Legendre Coefficients

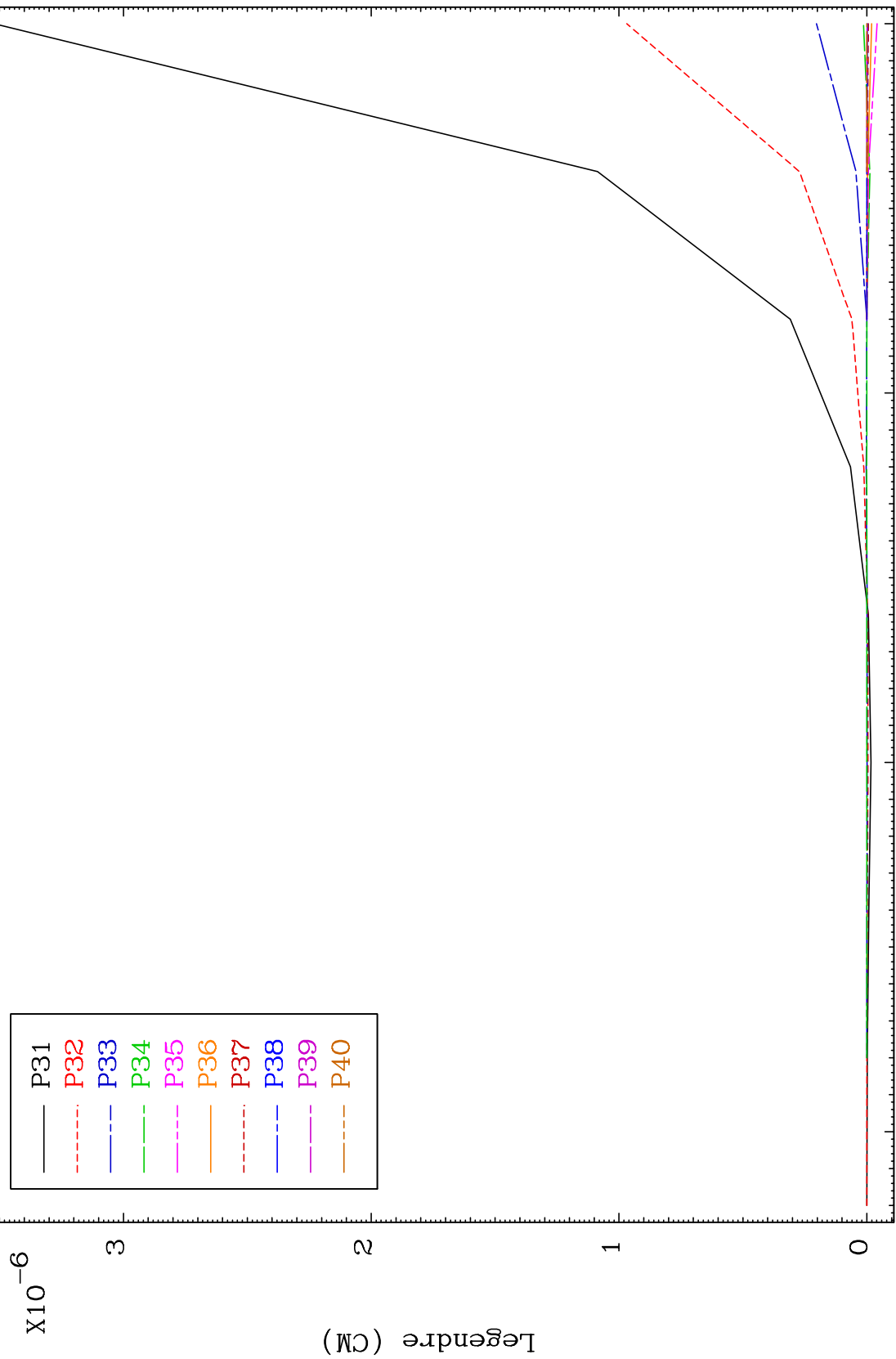
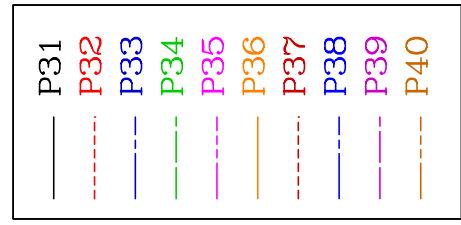
102-No-257



MAT 9975

Elastic Legendre Coefficients

102-No-257



22

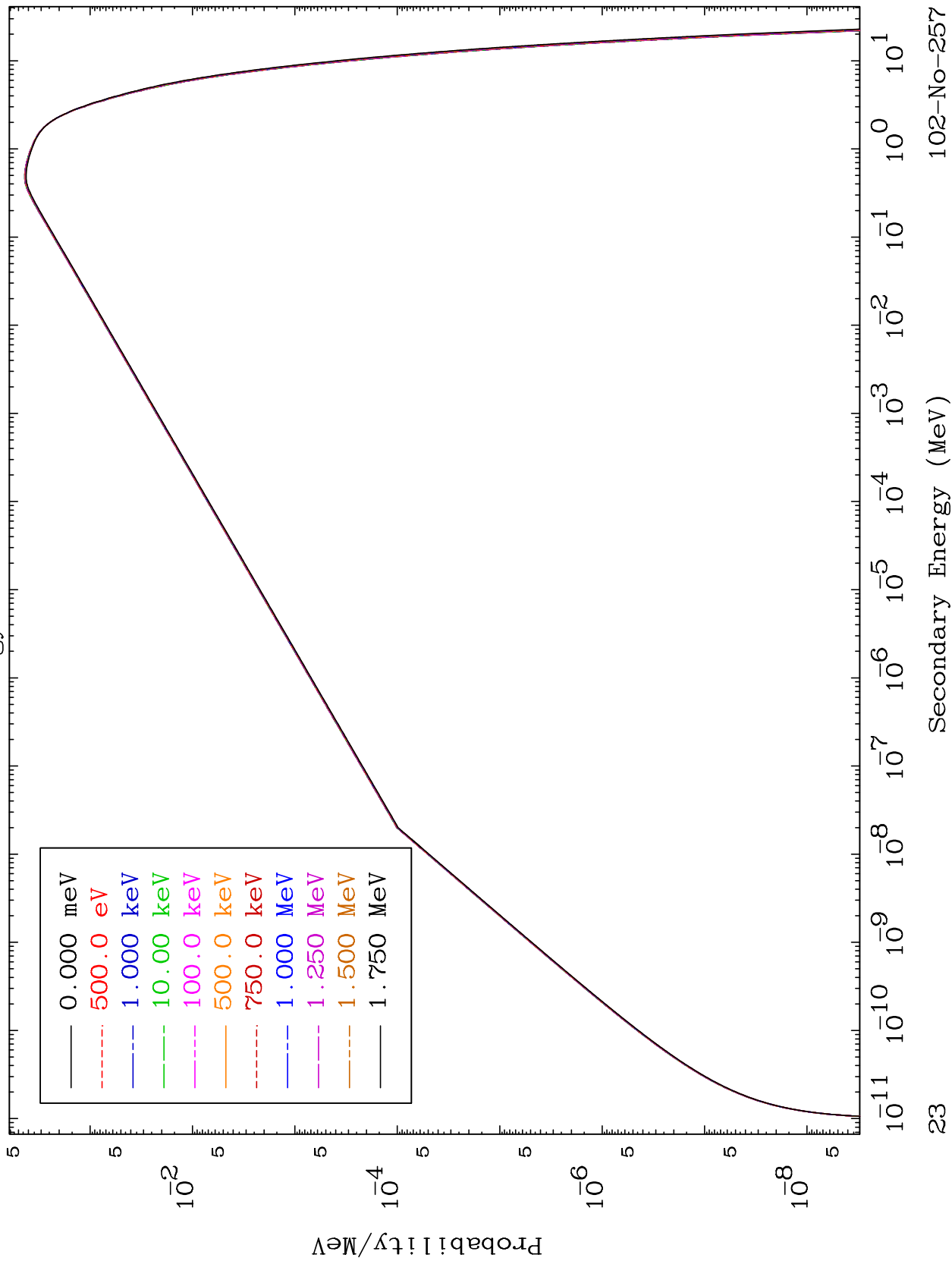
Incident Energy (MeV)

102-No-257

MAT 9975

### Fission Energy Distributions

102-No-257

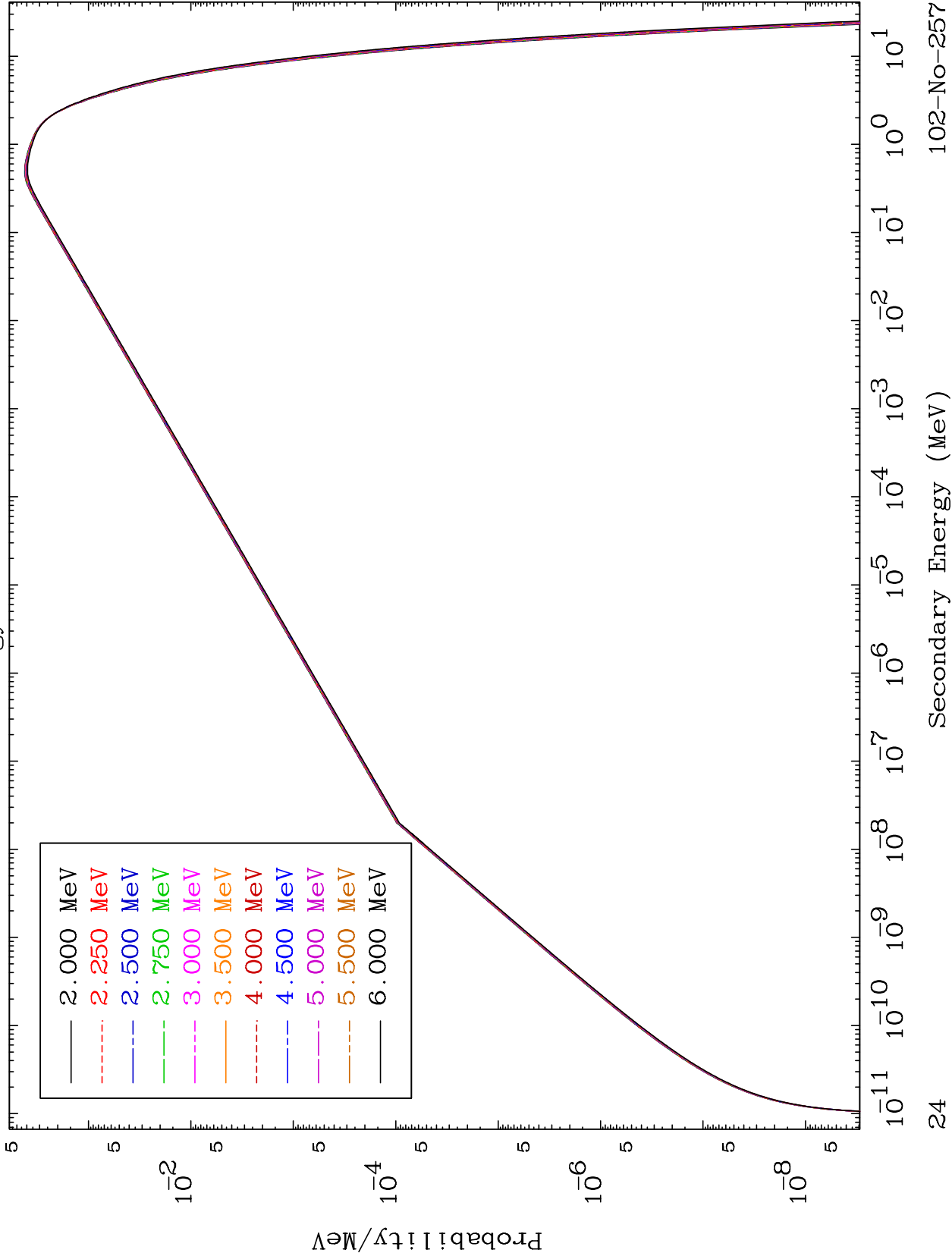


102-No-257

Secondary Energy (MeV)



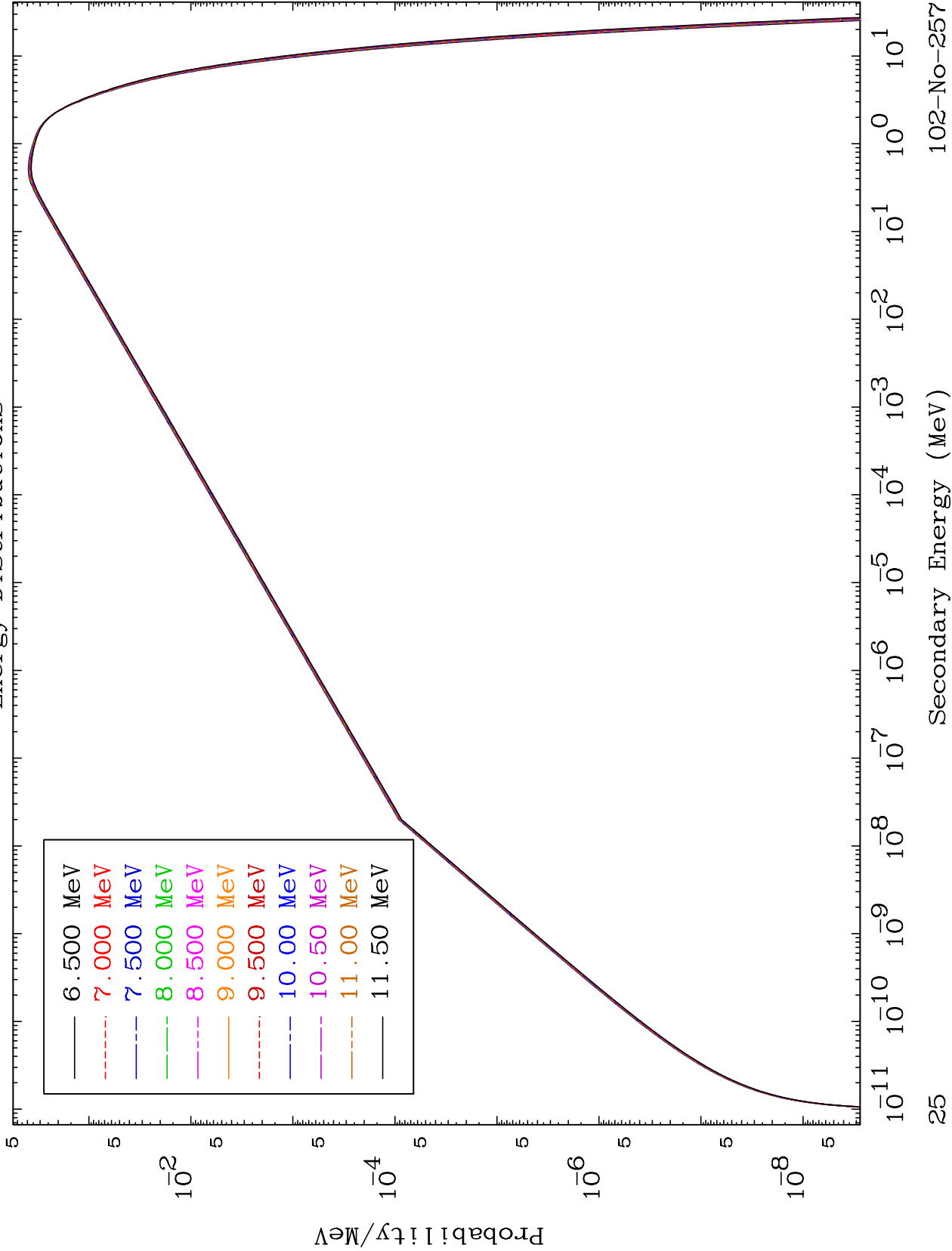
### Fission Energy Distributions



MAT 9975

### Fission Energy Distributions

102-No-257

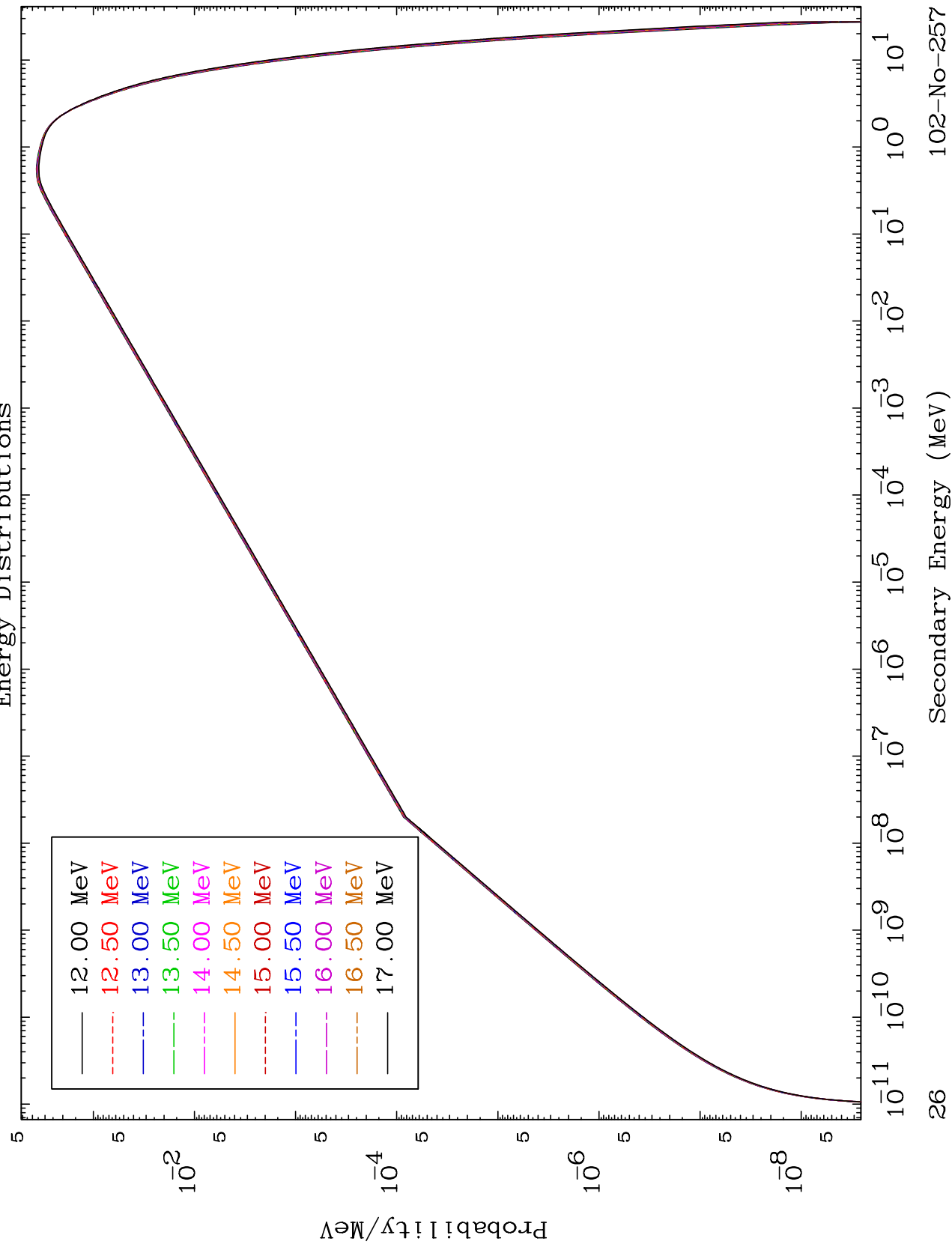


25

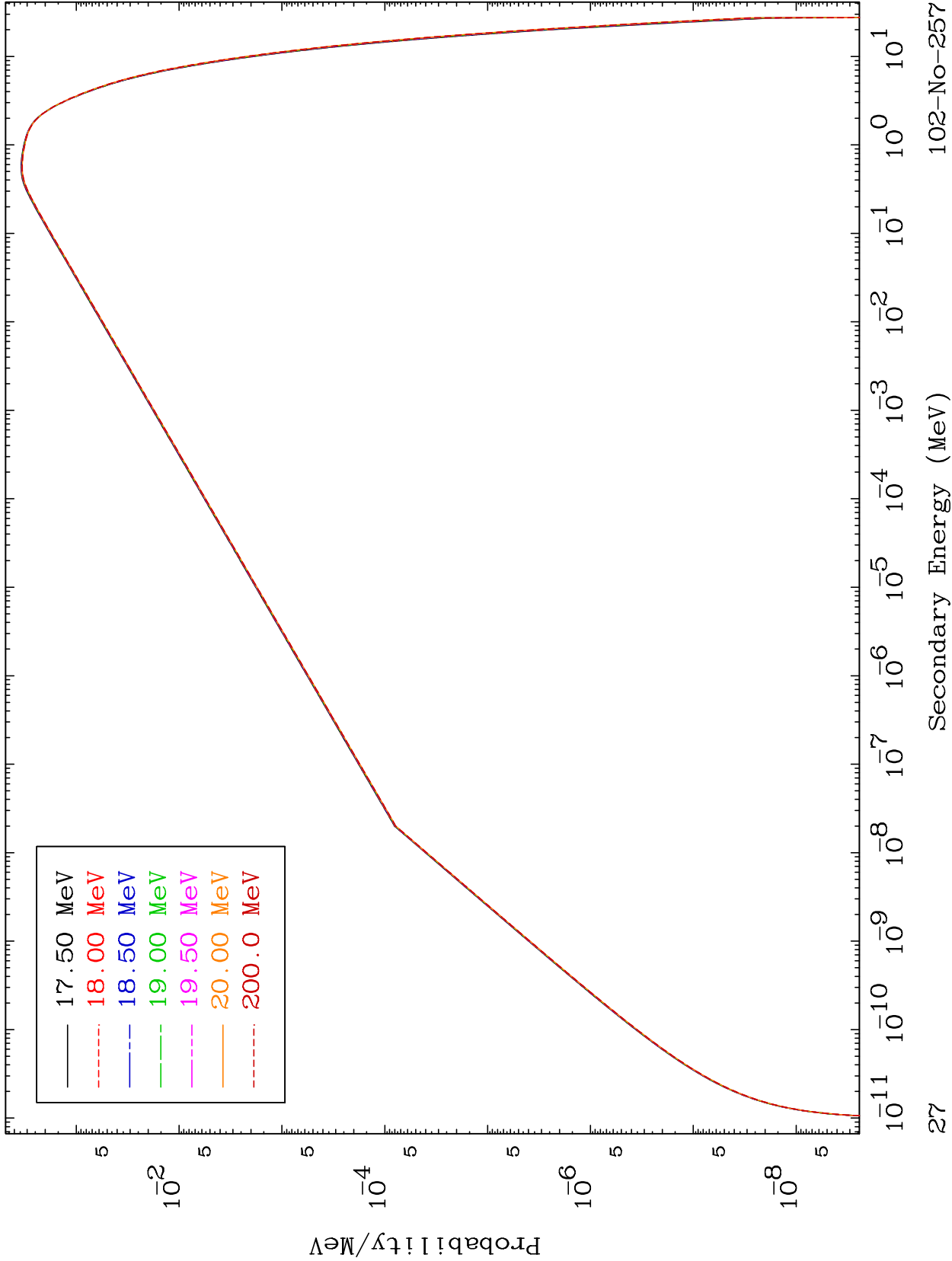
MAT 9975

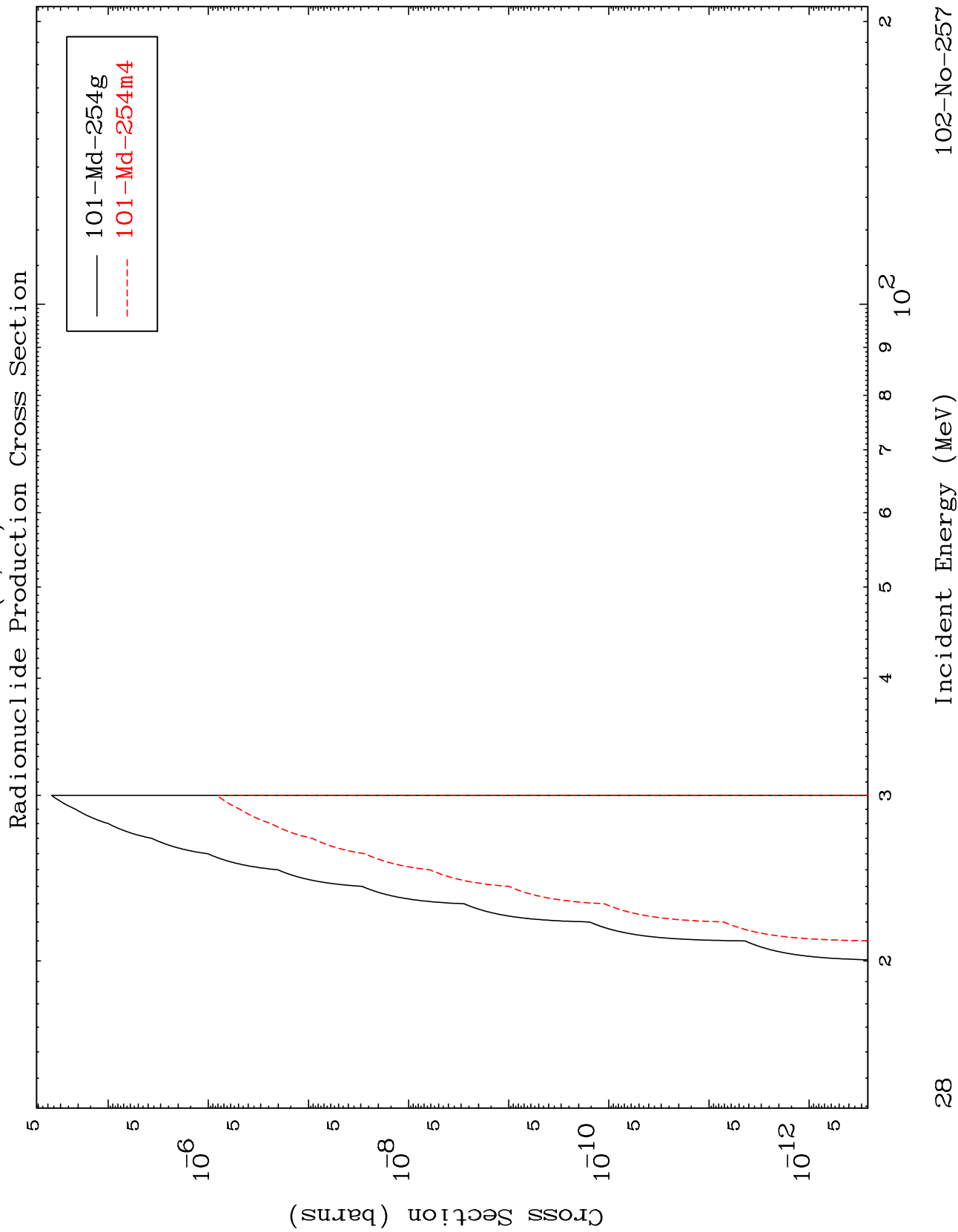
Fission Energy Distributions

102-No-257



Fission Energy Distributions



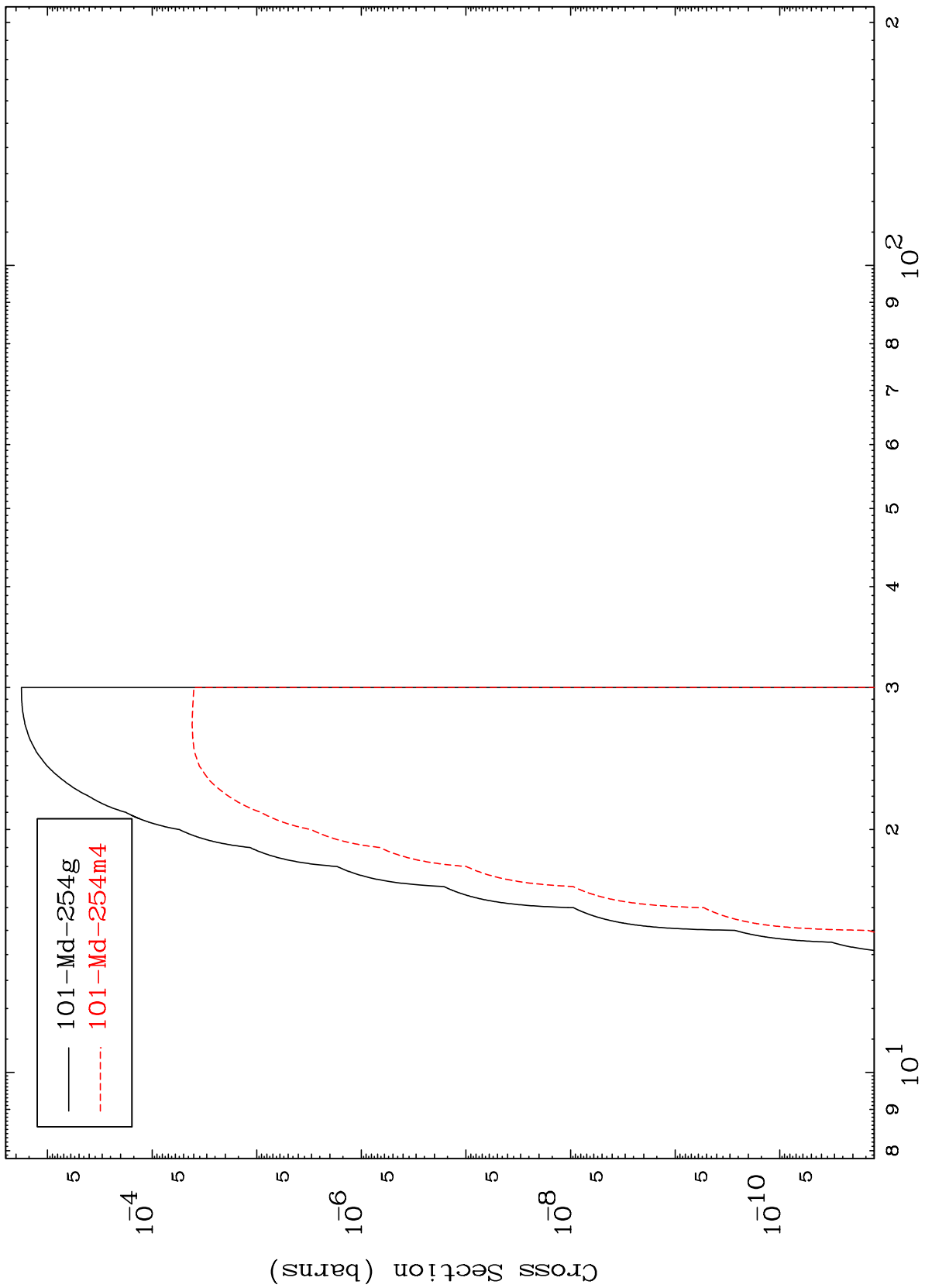


MAT 9975

(n,n') t

102-No-257

Radionuclide Production Cross Section



29

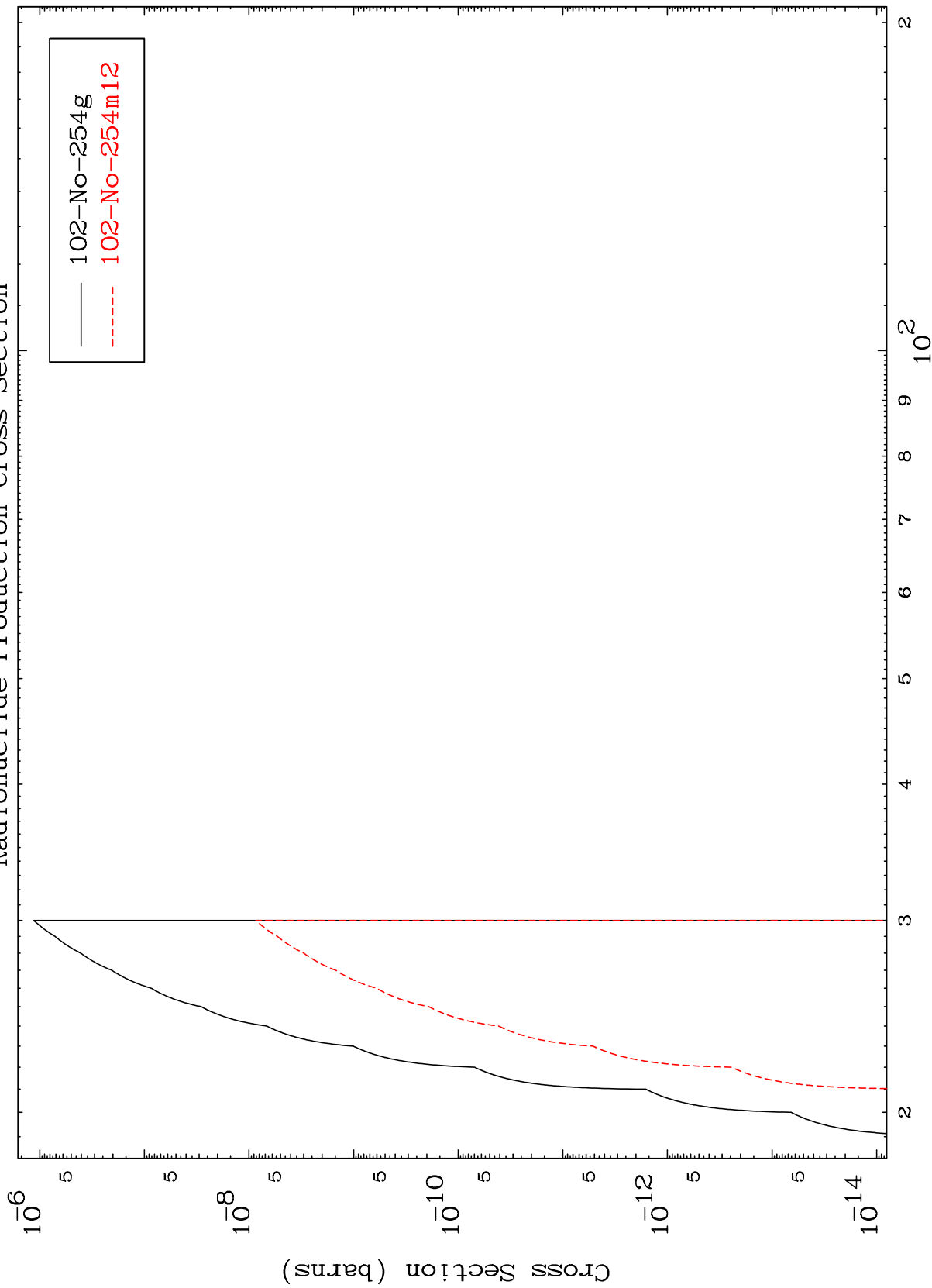
Incident Energy (MeV)

102-No-257

MAT 9975

102-No-257

(n,4n)  
Radionuclide Production Cross Section



102-No-257

Incident Energy (MeV)

30

Radionuclide Production Cross Section

