

Program EVALPLOT
(Version 2021-1)

by

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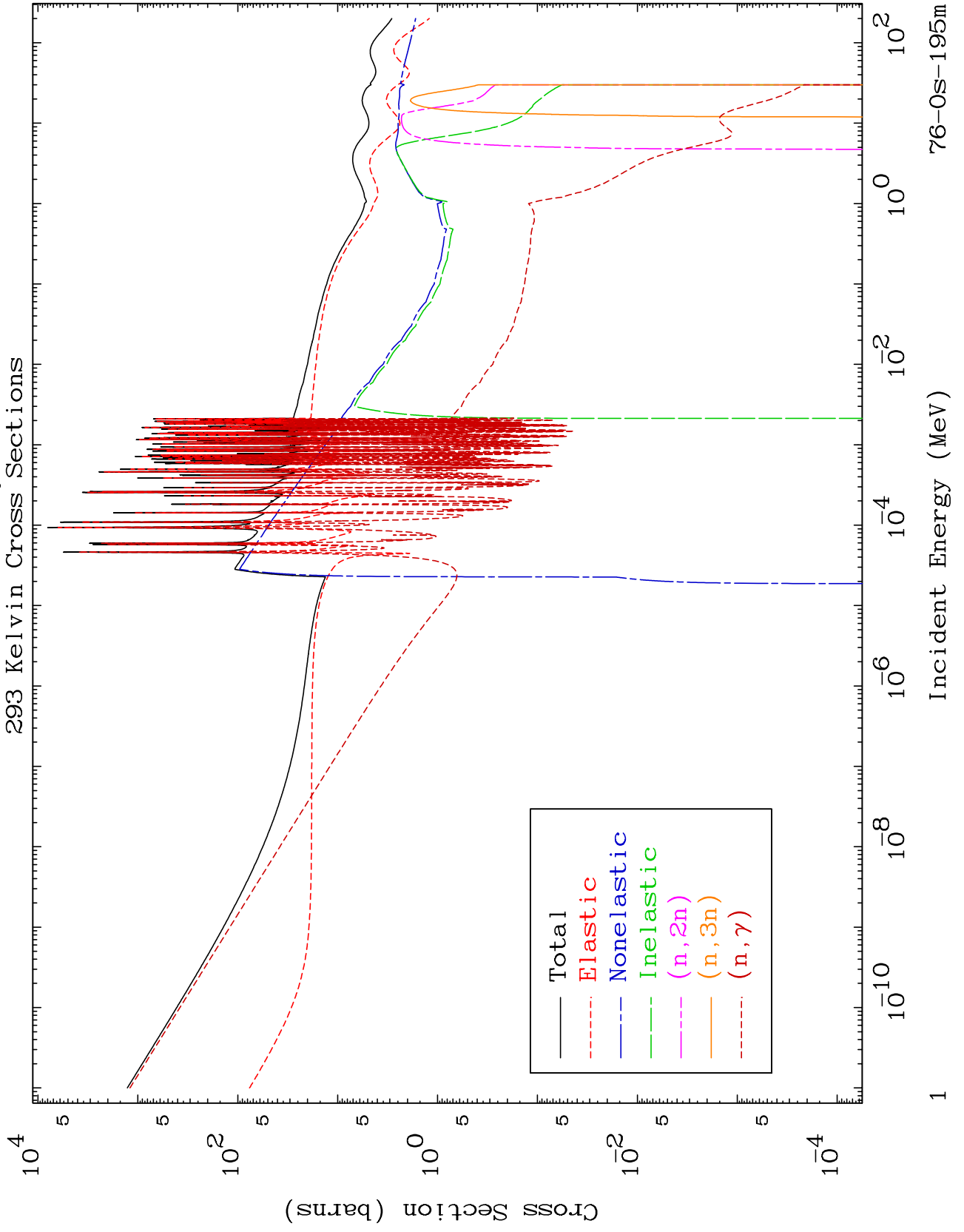
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 7659

Neutron Major
293 Kelvin Cross Sections

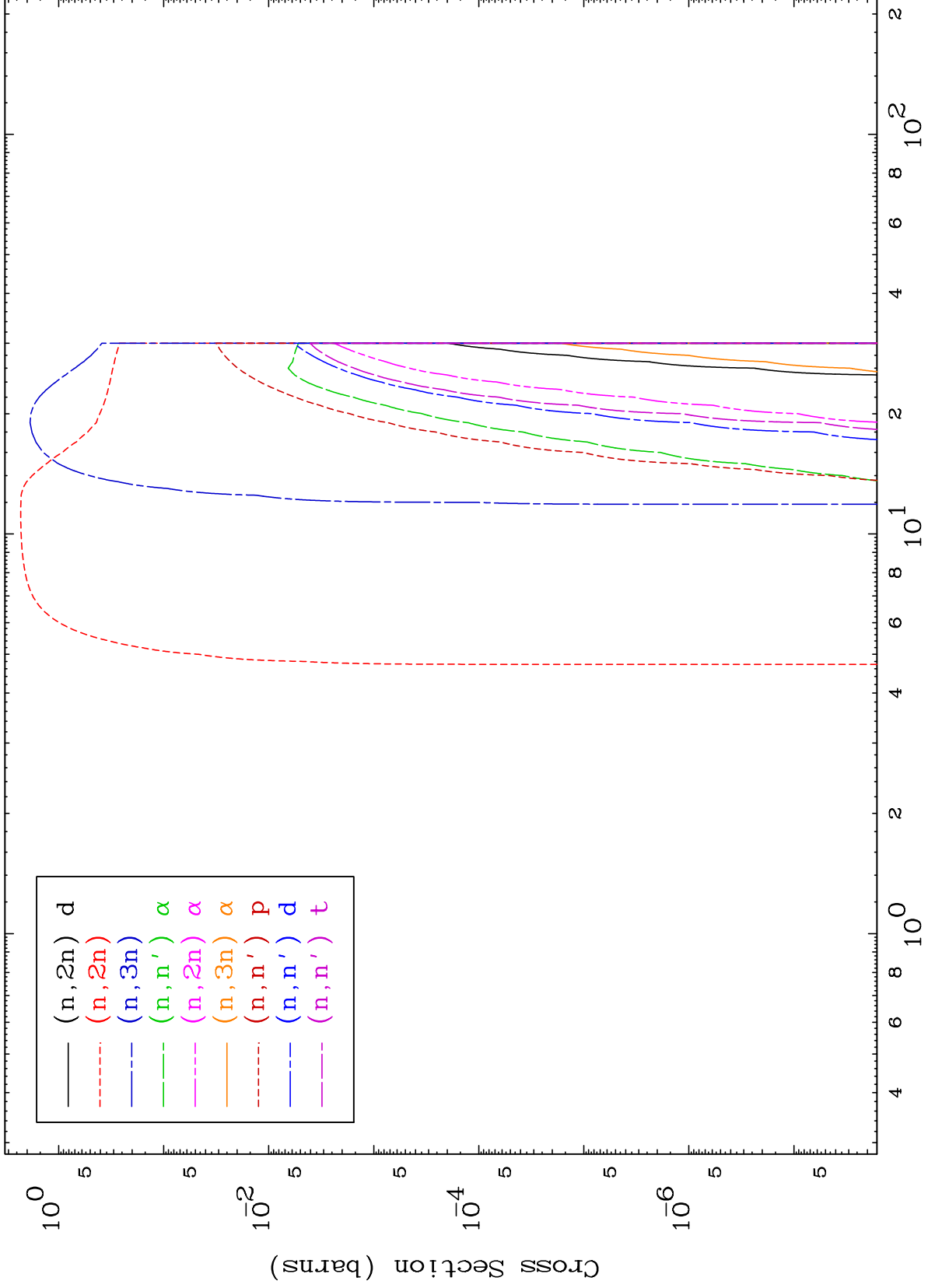
76-0s-195m



MAT 7659

Neutron Absorption
293 Kelvin Cross Sections

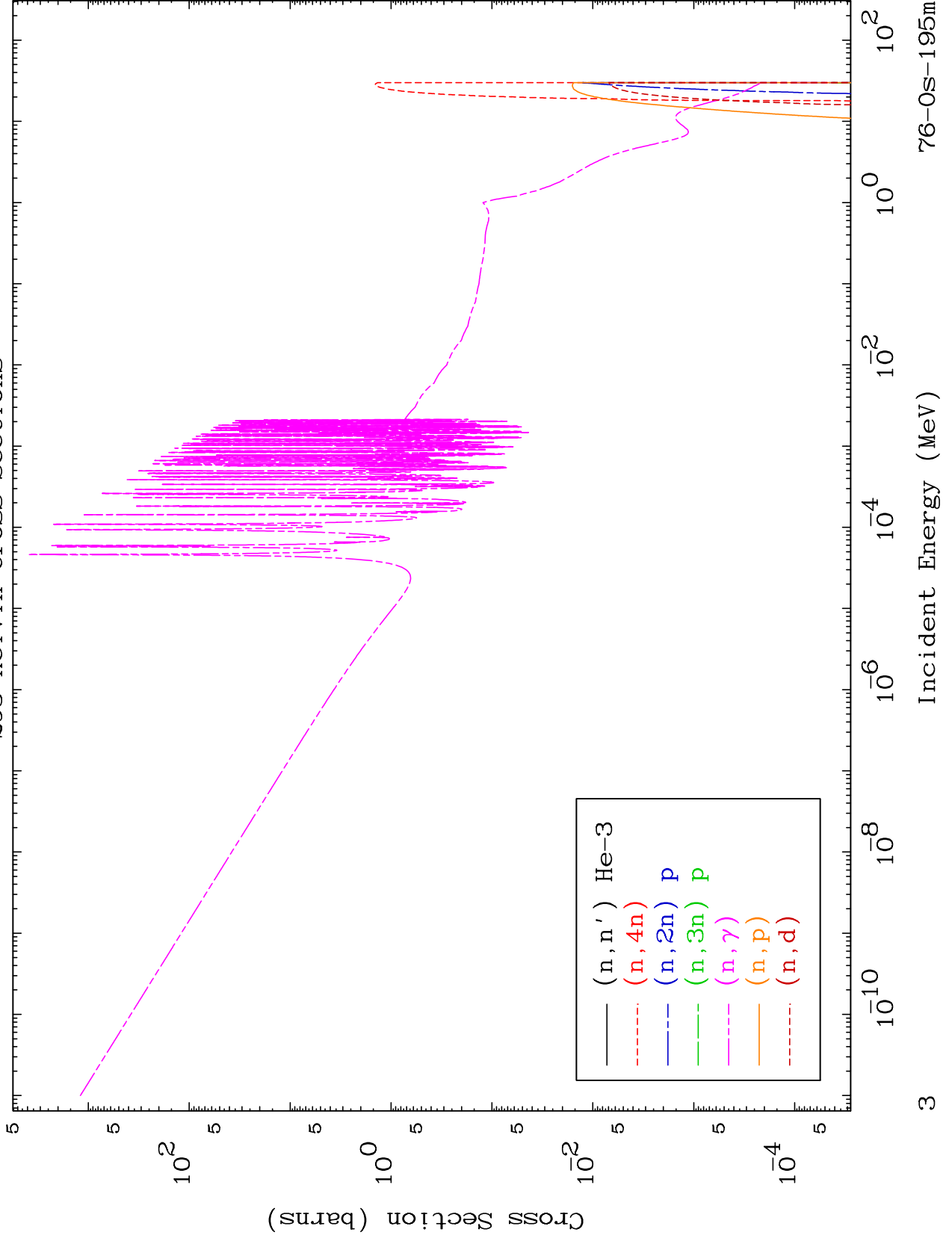
76-0s-195m



MAT 7659

Neutron Absorption
293 Kelvin Cross Sections

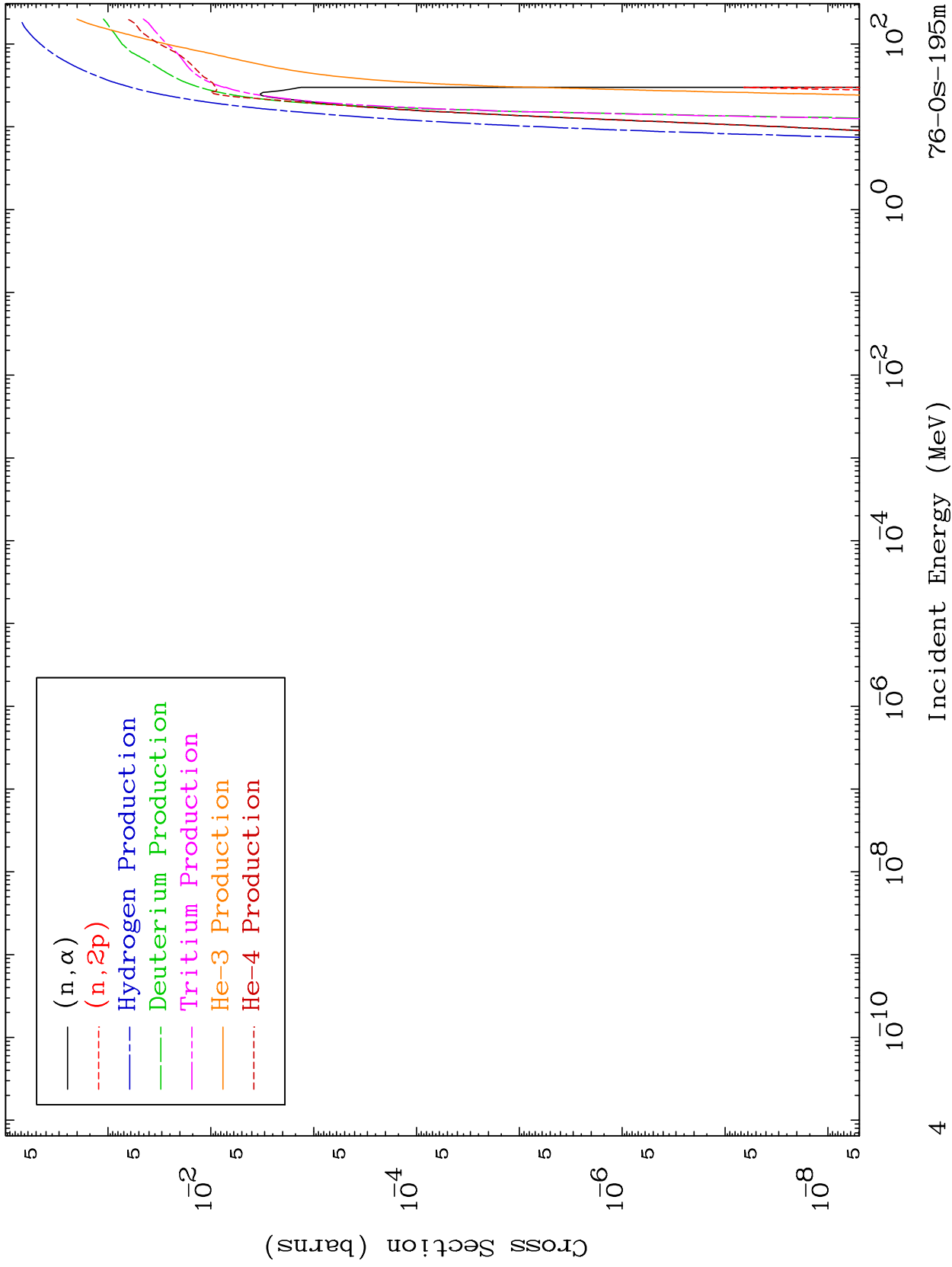
76-0s-195m



MAT 7659

Neutron Absorption
293 Kelvin Cross Sections

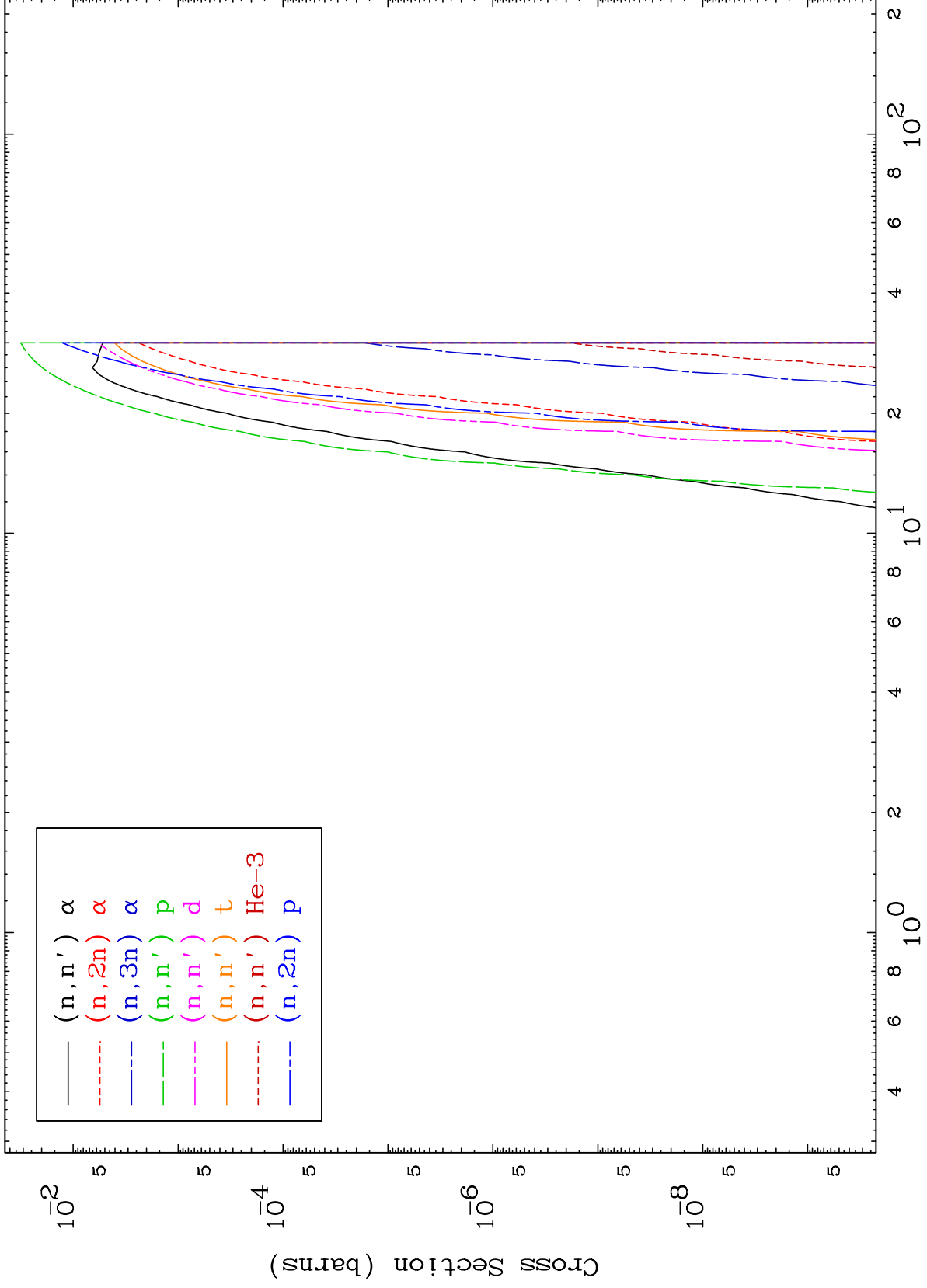
76-0s-195m



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Charged Particle
293 Kelvin Cross Sections

76-0s-195m



5

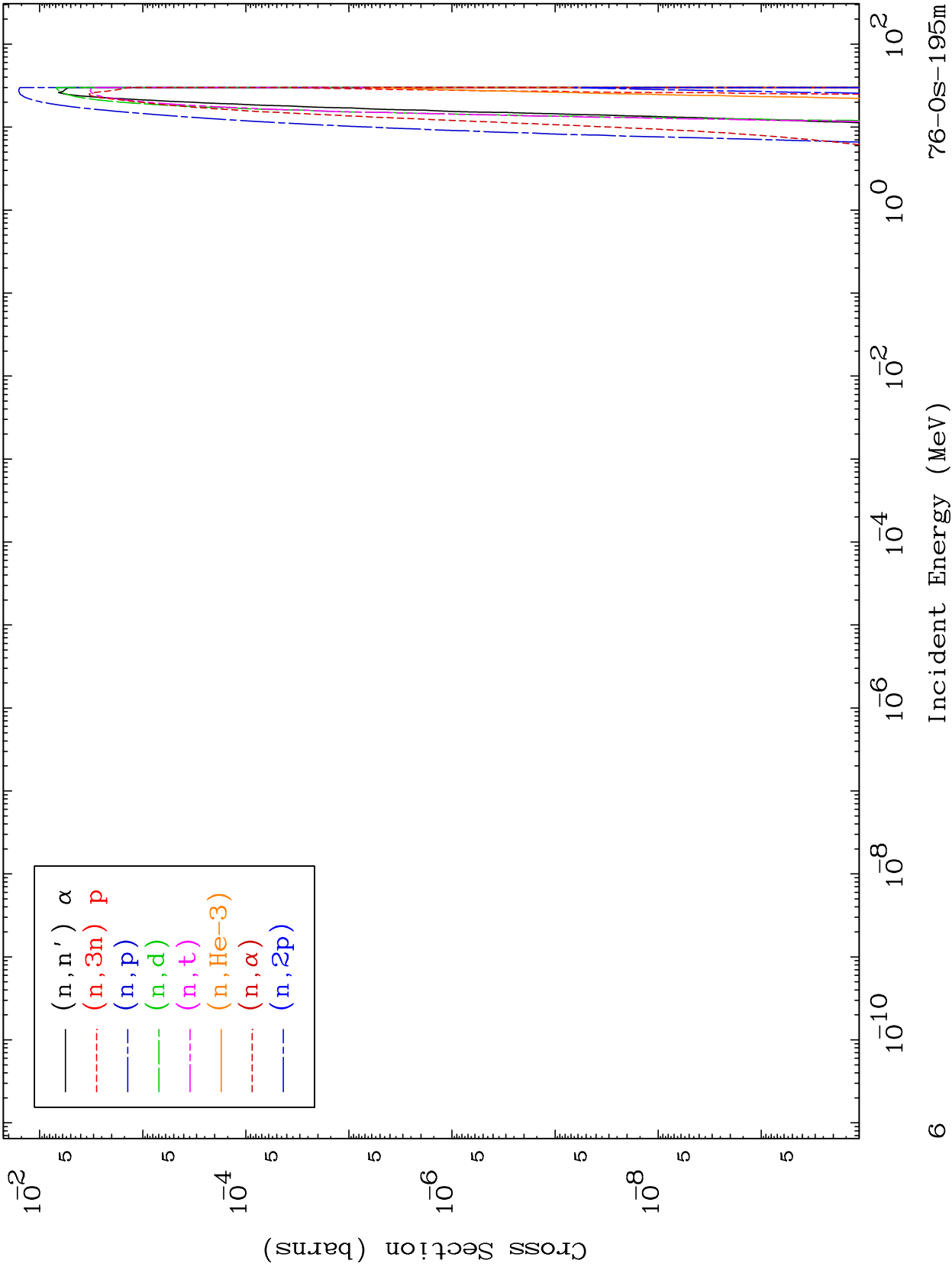
Incident Energy (MeV)

76-0s-195m

MAT 7659

Charged Particle
293 Kelvin Cross Sections

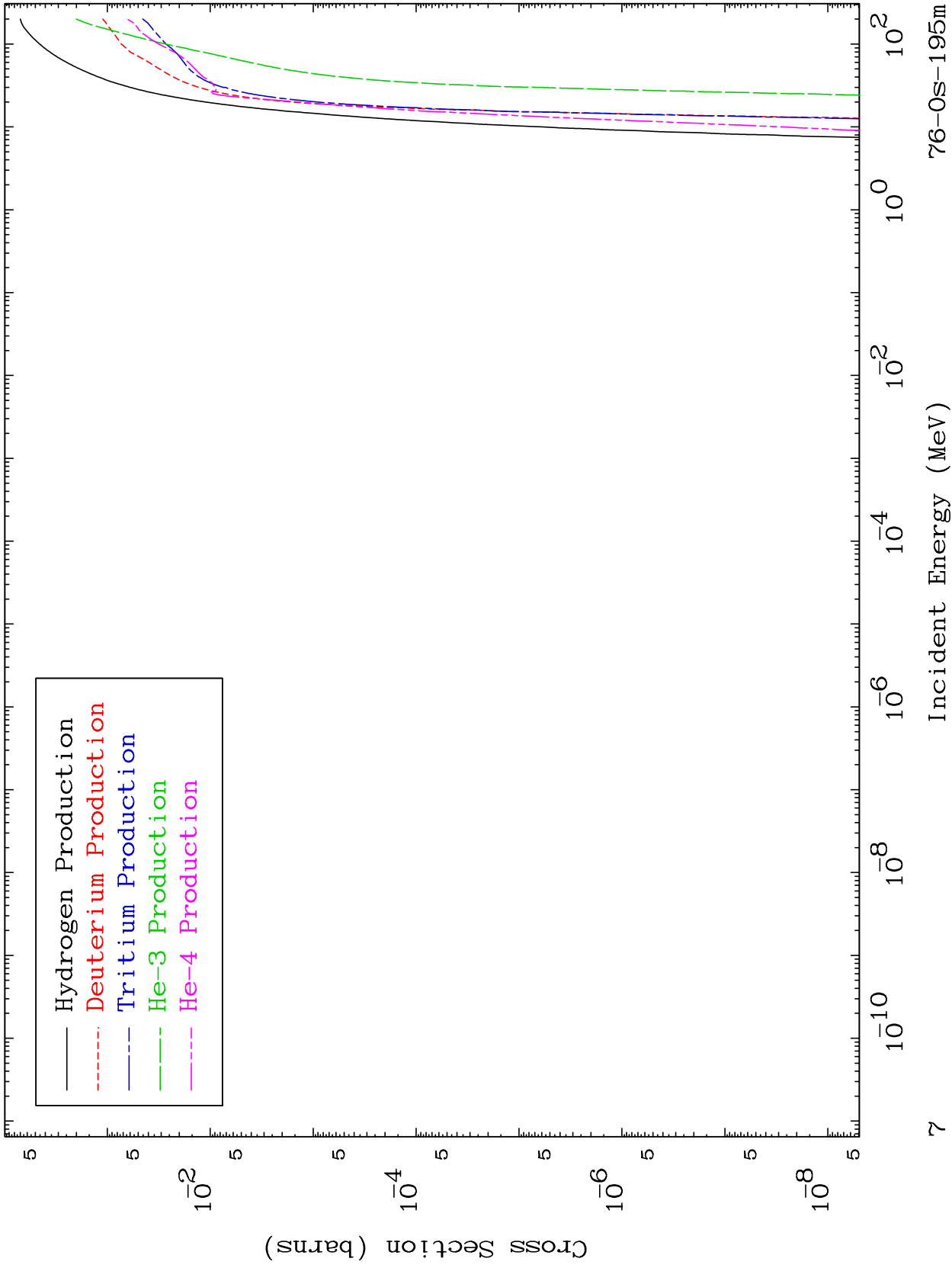
76-0s-195m



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Particle Production
293 Kelvin Cross Sections

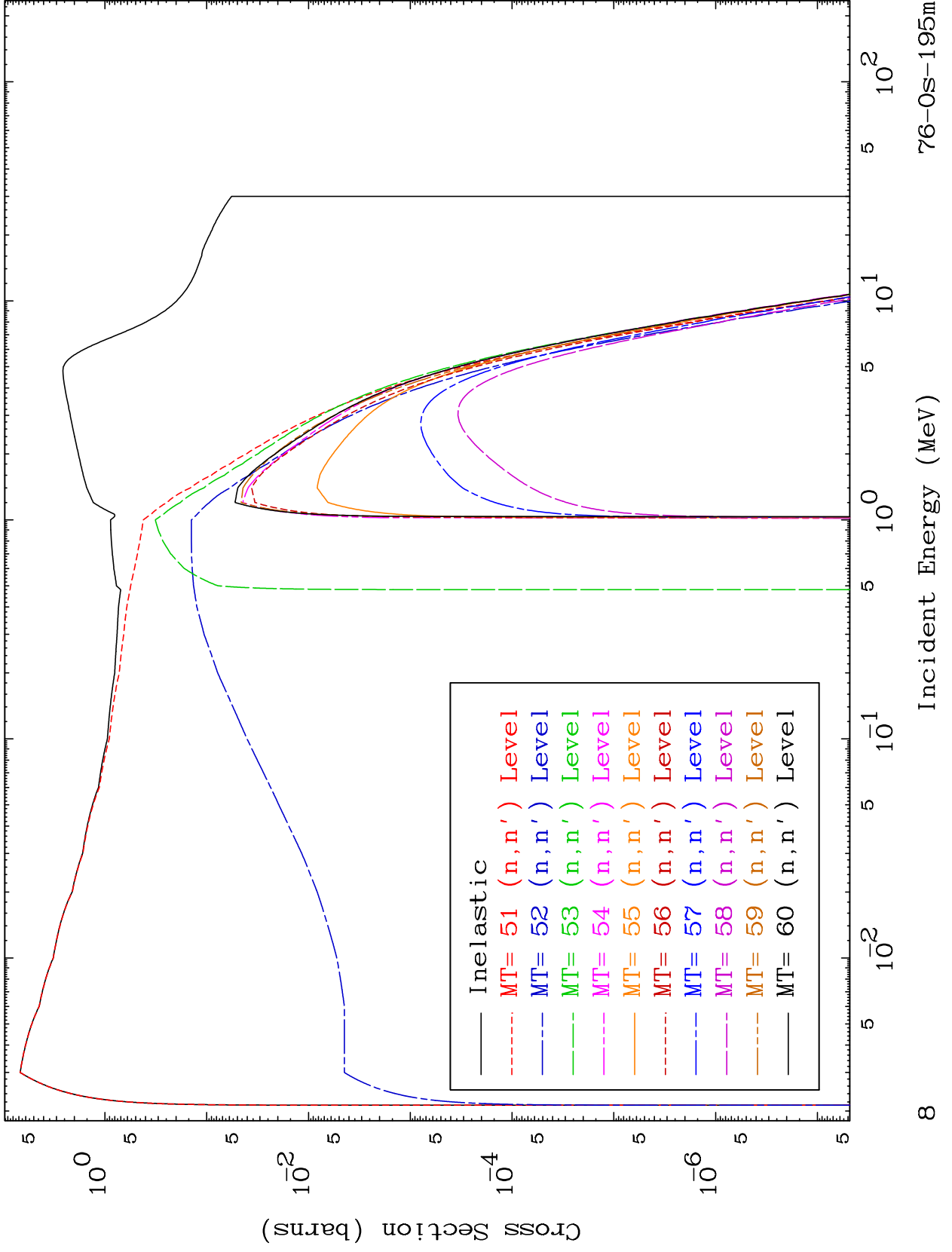
76-0s-195m



MAT 7659

(n,n') Levels
293 Kelvin Cross Sections

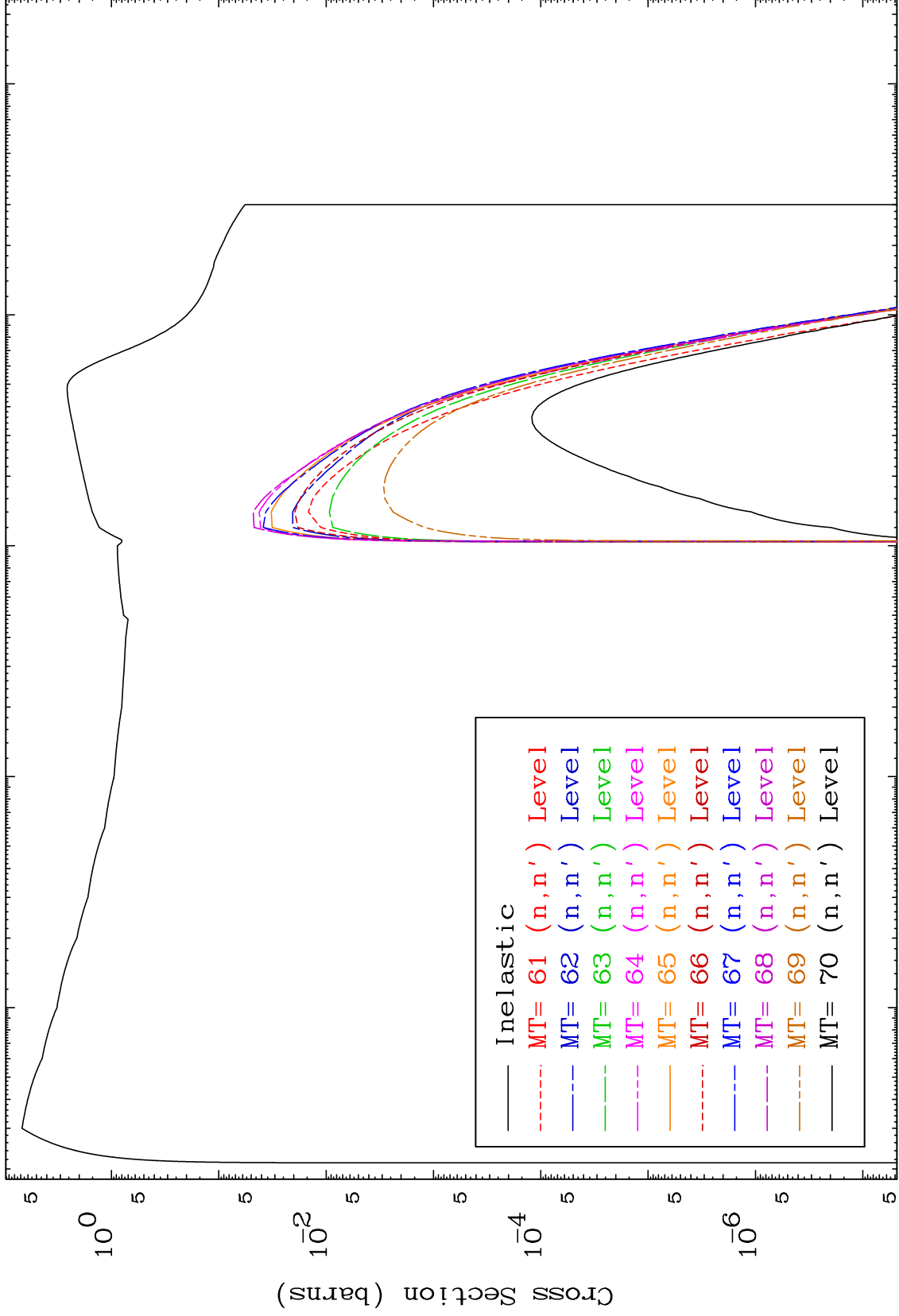
76-0s-195m



MAT 7659

(n,n') Levels
293 Kelvin Cross Sections

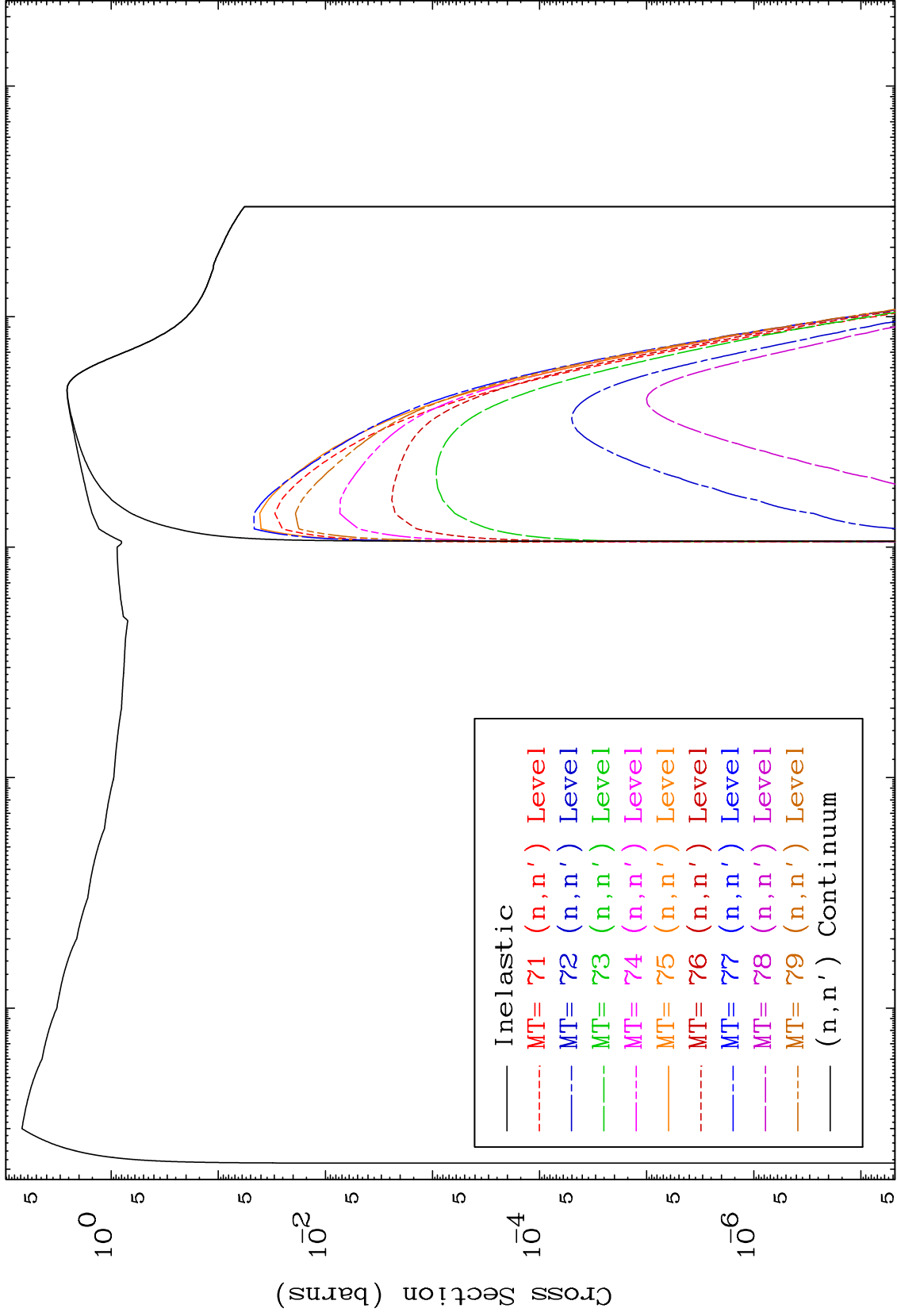
76-0s-195m



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(n,n') Levels
293 Kelvin Cross Sections

76-0s-195m



10

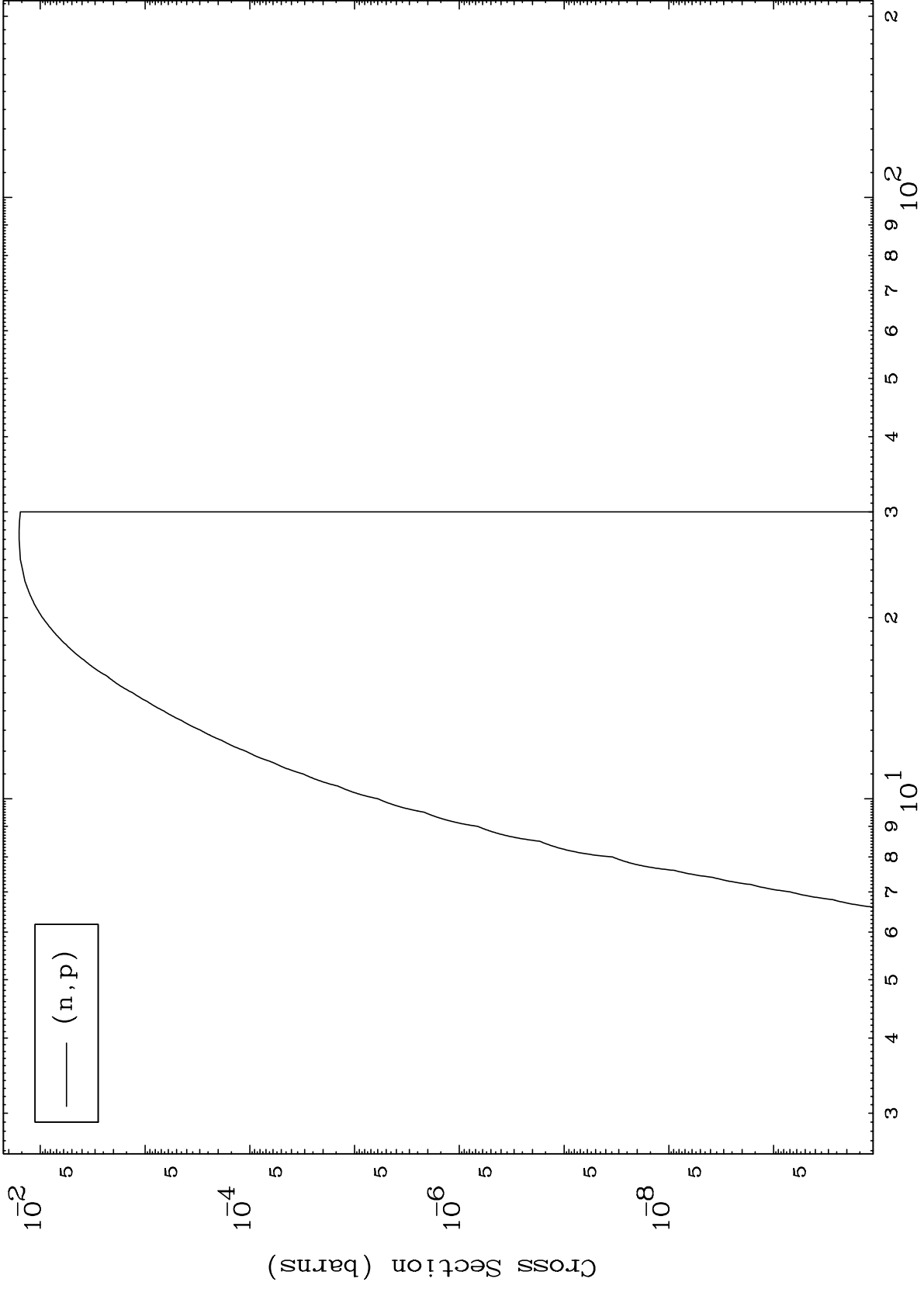
Incident Energy (MeV)

76-0s-195m

MAT 7659

(n,p) Levels
293 Kelvin Cross Sections

76-0s-195m



(n,p)

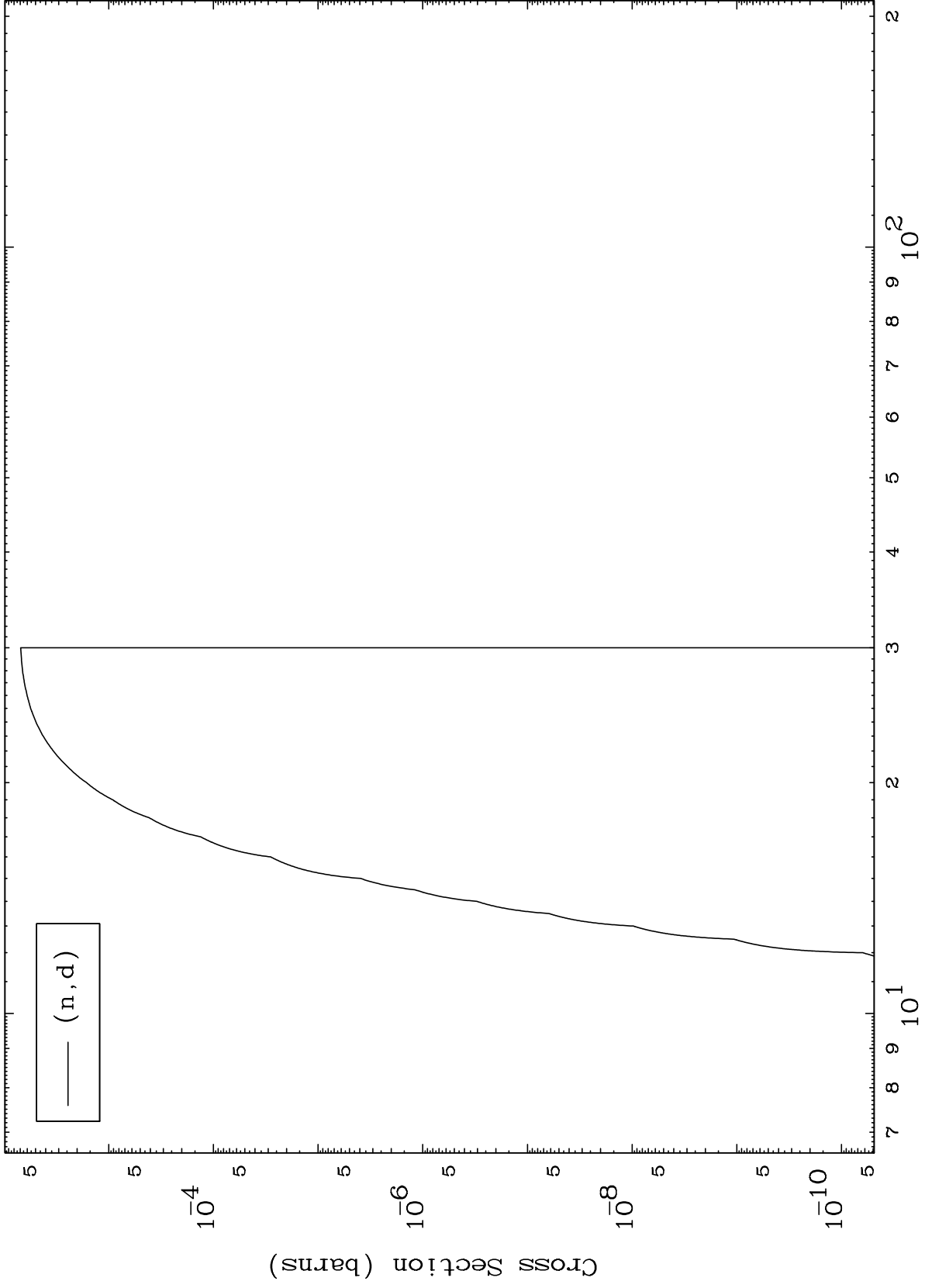
Incident Energy (MeV)

76-0s-195m

MAT 7659

(n,d) Levels
293 Kelvin Cross Sections

76-0s-195m



12

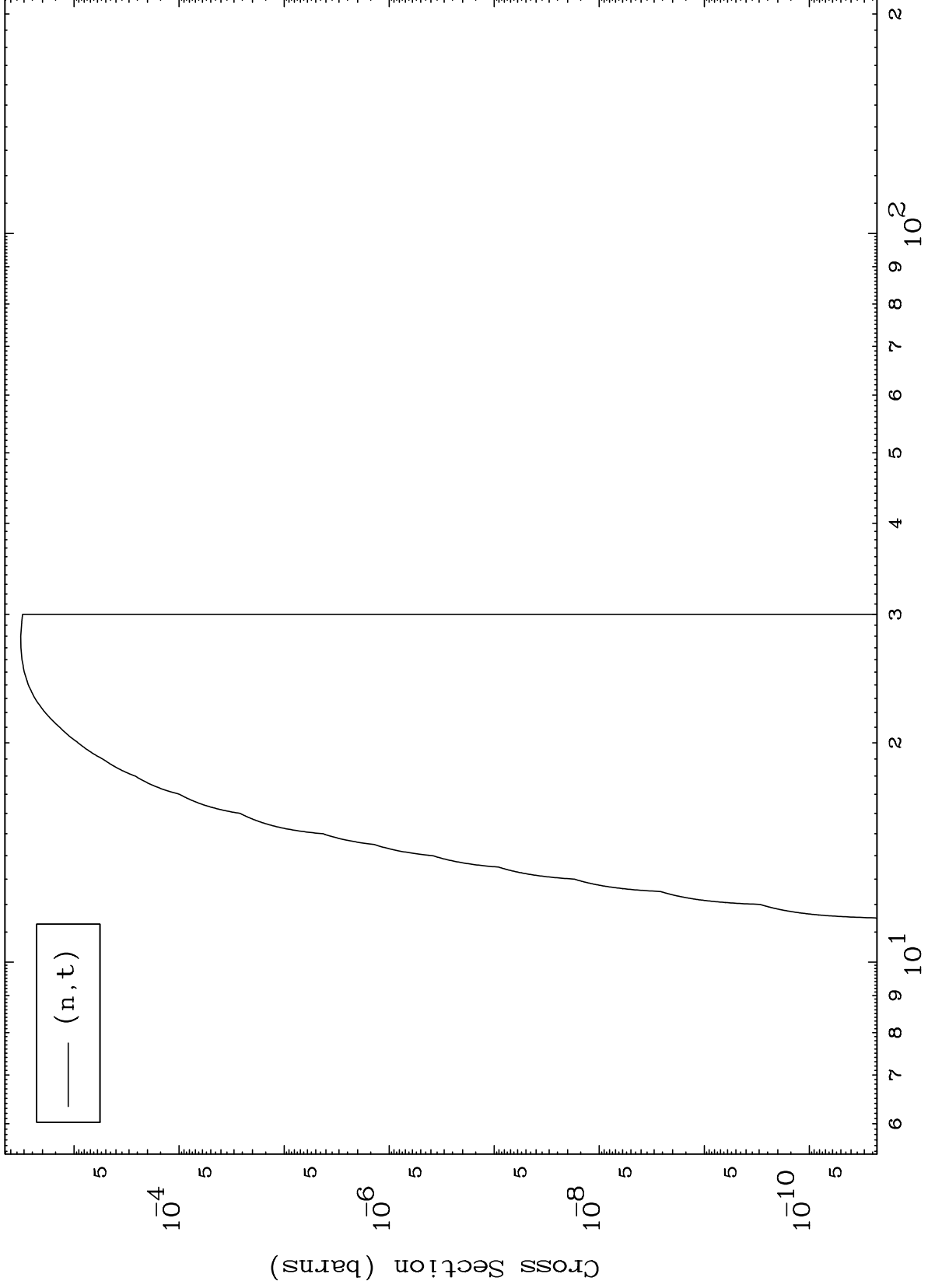
Incident Energy (MeV)

76-0s-195m

MAT 7659

(n,t) Levels
293 Kelvin Cross Sections

76-0s-195m



13

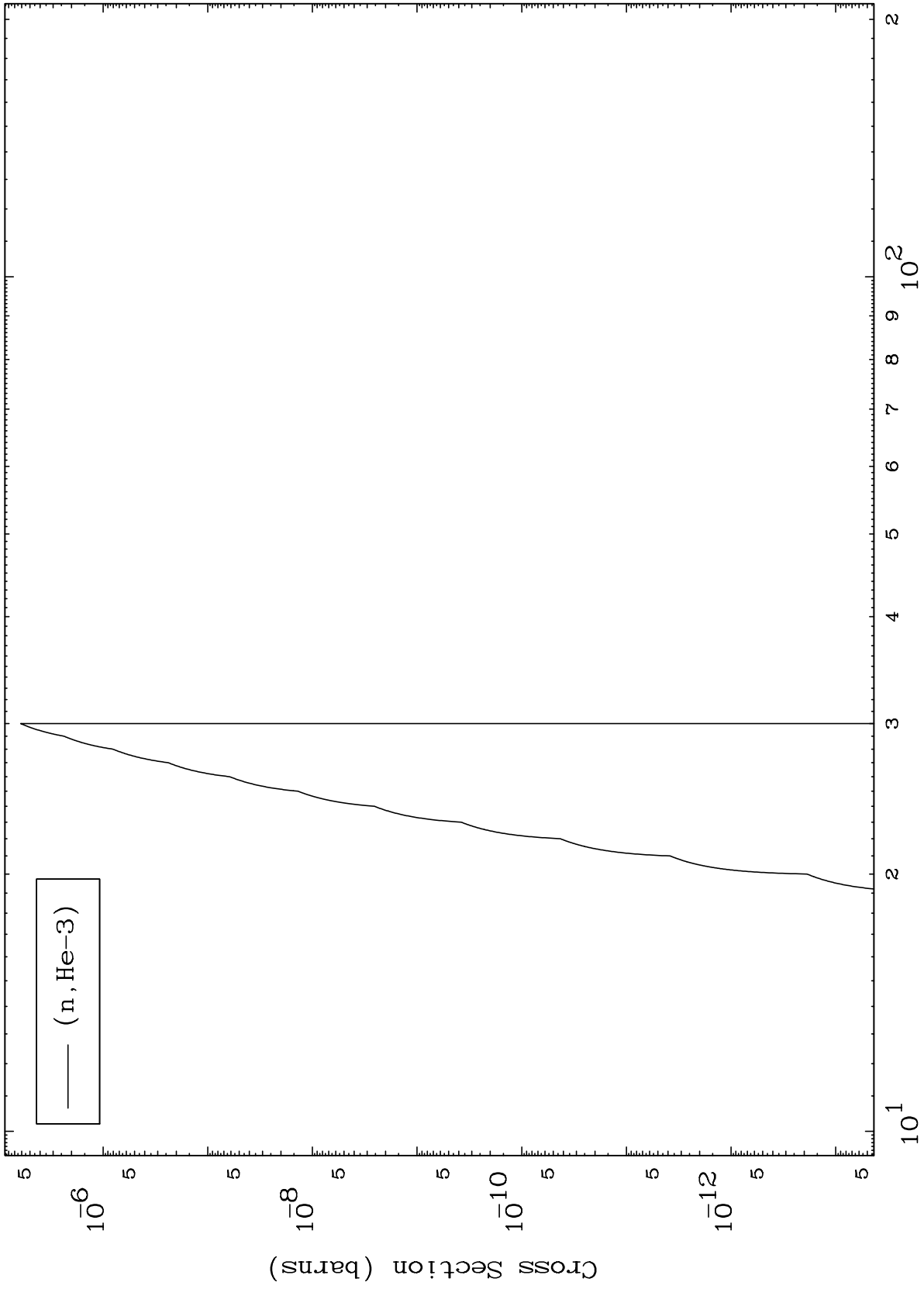
Incident Energy (MeV)

76-0s-195m

MAT 7659

(n,He3) Levels
293 Kelvin Cross Sections

76-0s-195m



14

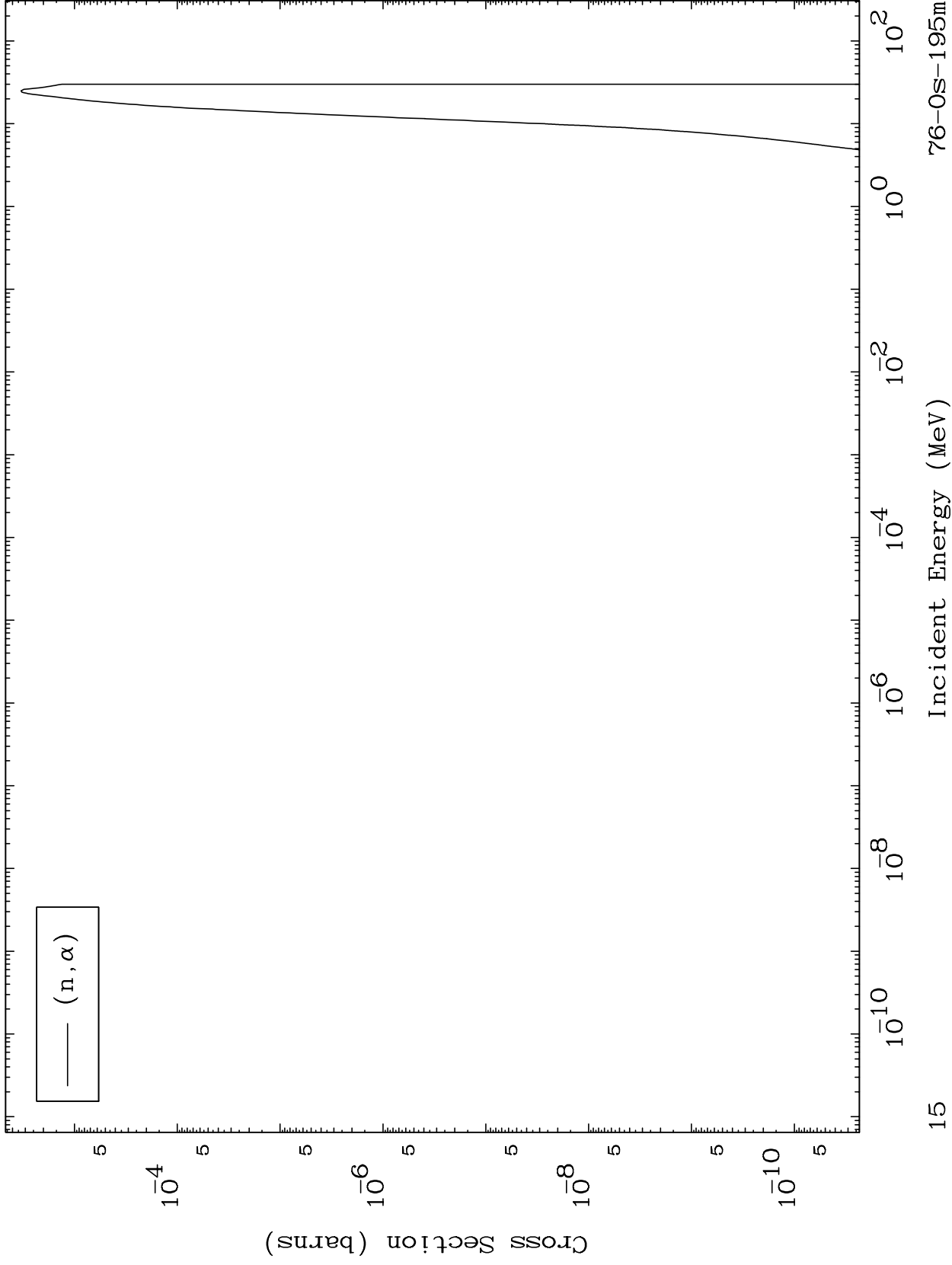
Incident Energy (MeV)

76-0s-195m

MAT 7659

(n, α) Levels
293 Kelvin Cross Sections

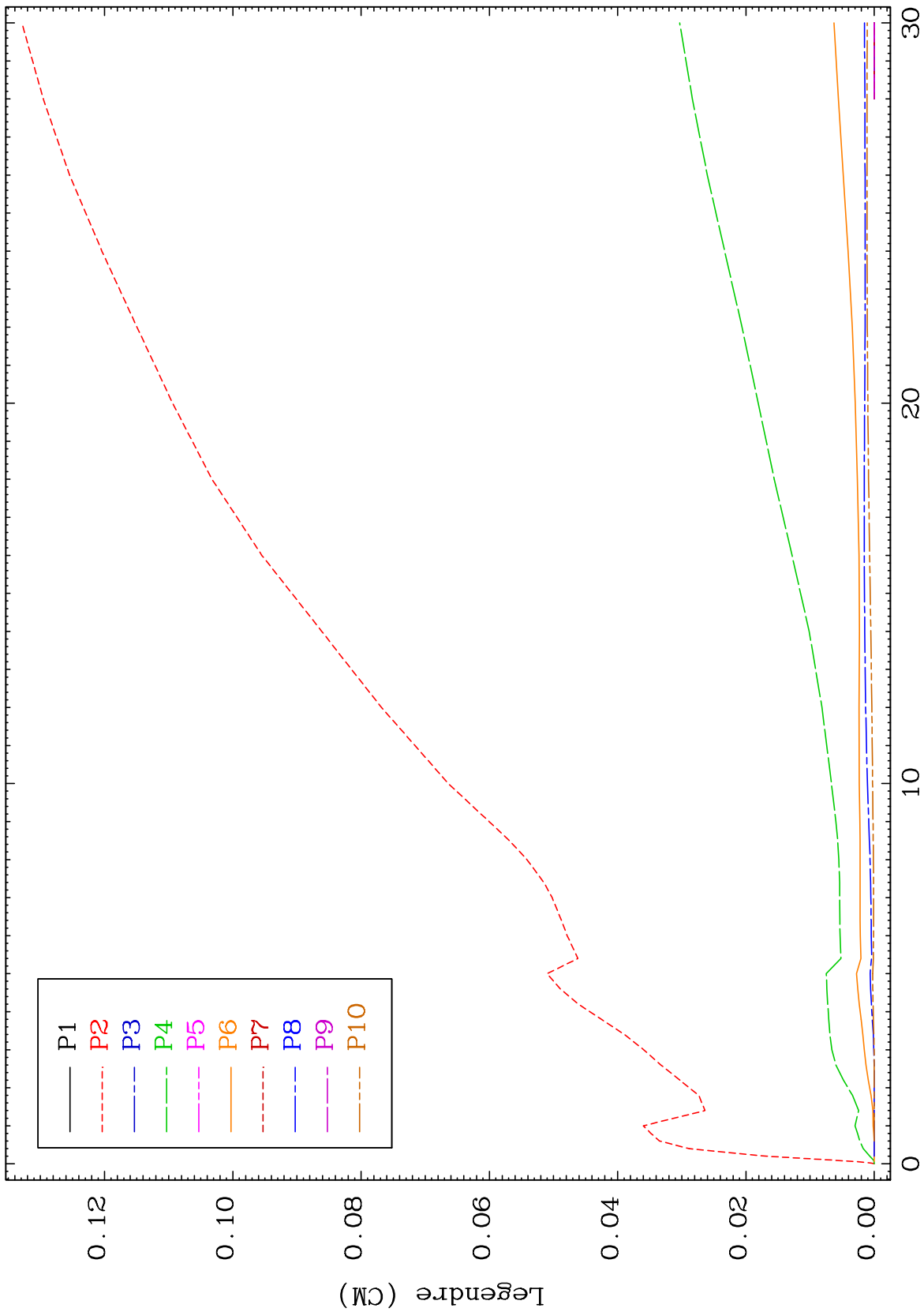
76-0s-195m



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Elastic Legendre Coefficients

76-0s-195m



76-0s-195m

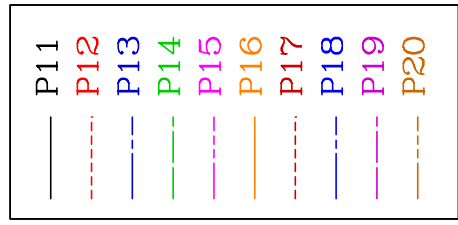
Incident Energy (MeV)

16

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Elastic Legendre Coefficients

76-0s-195m



$\times 10^{-4}$

Legendre (CM)

6

4

2

0

5

10

15

20

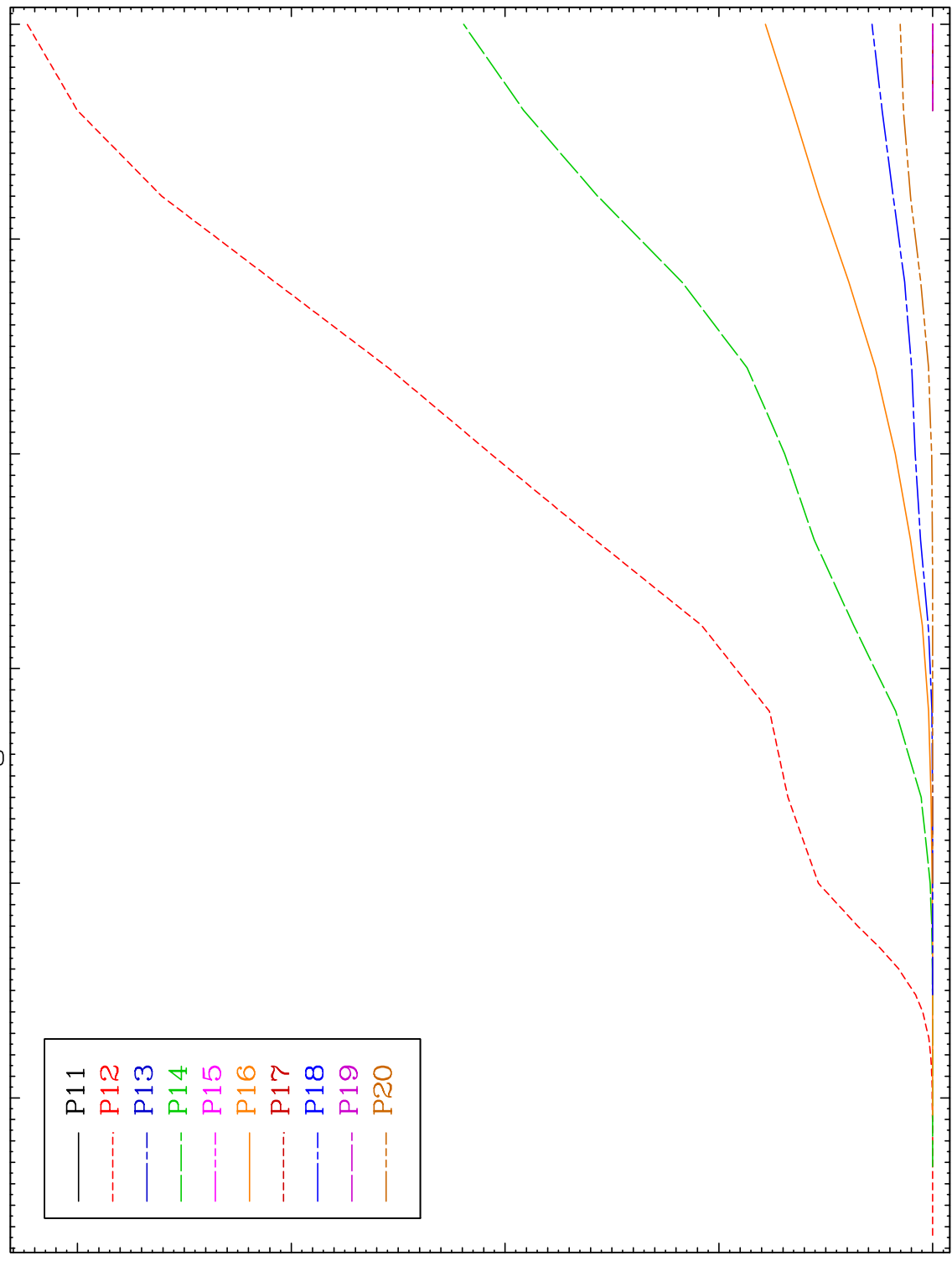
25

30

17

Incident Energy (MeV)

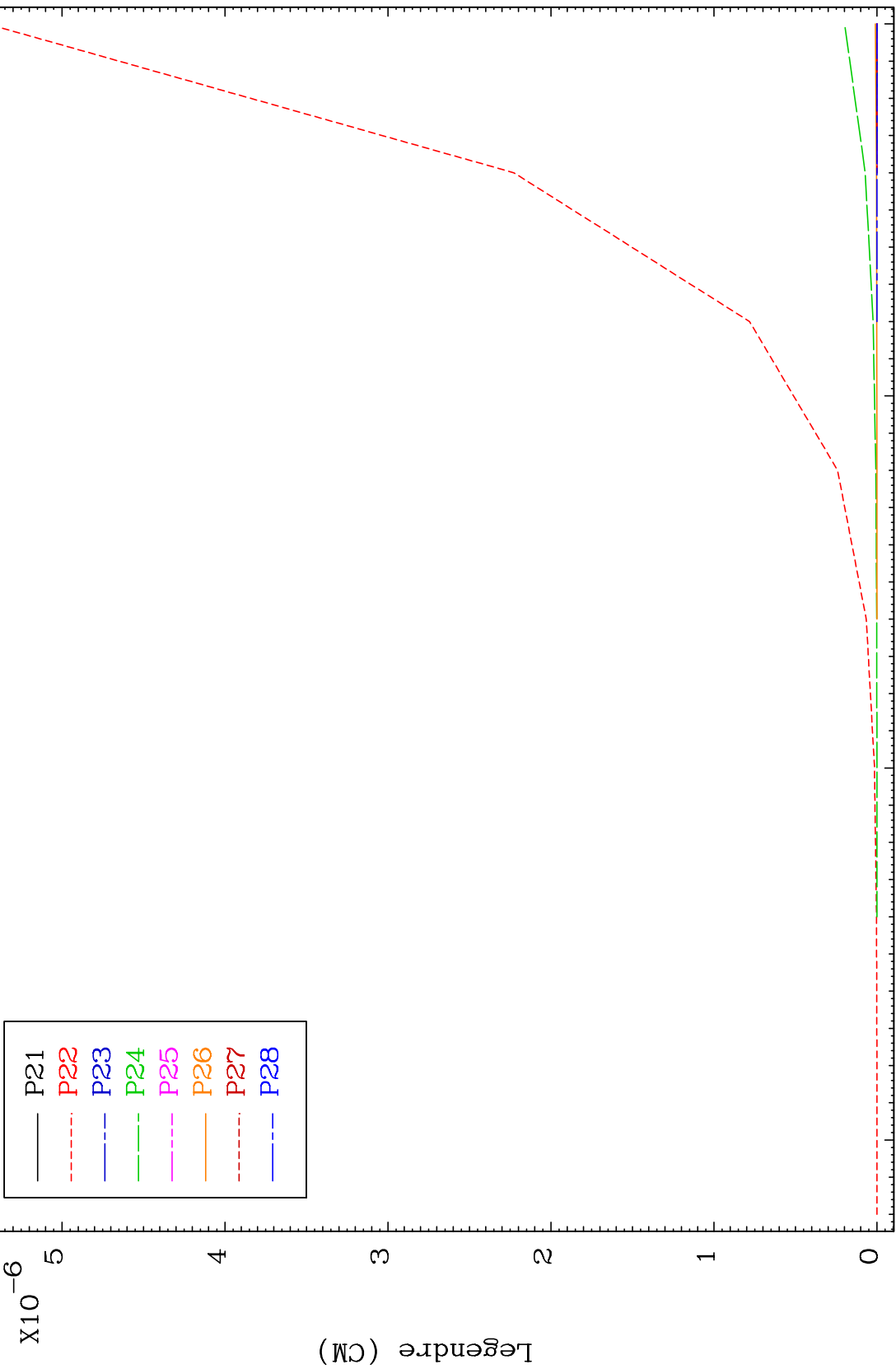
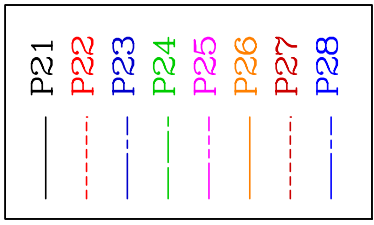
76-0s-195m



MAT 7659

Elastic Legendre Coefficients

76-0s-195m



18

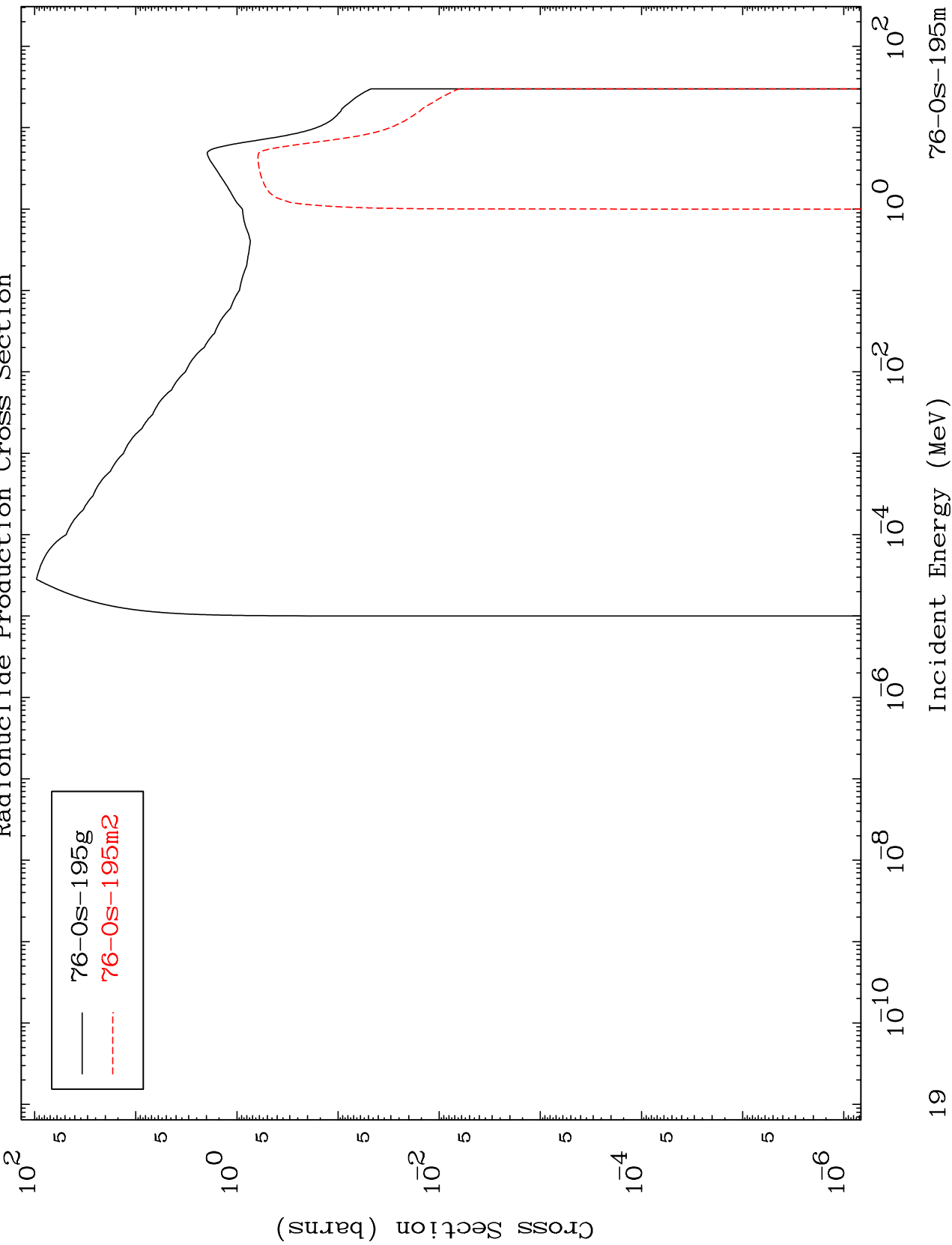
Incident Energy (MeV)

76-0s-195m

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Radionuclide Production Cross Section

76-0s-195m

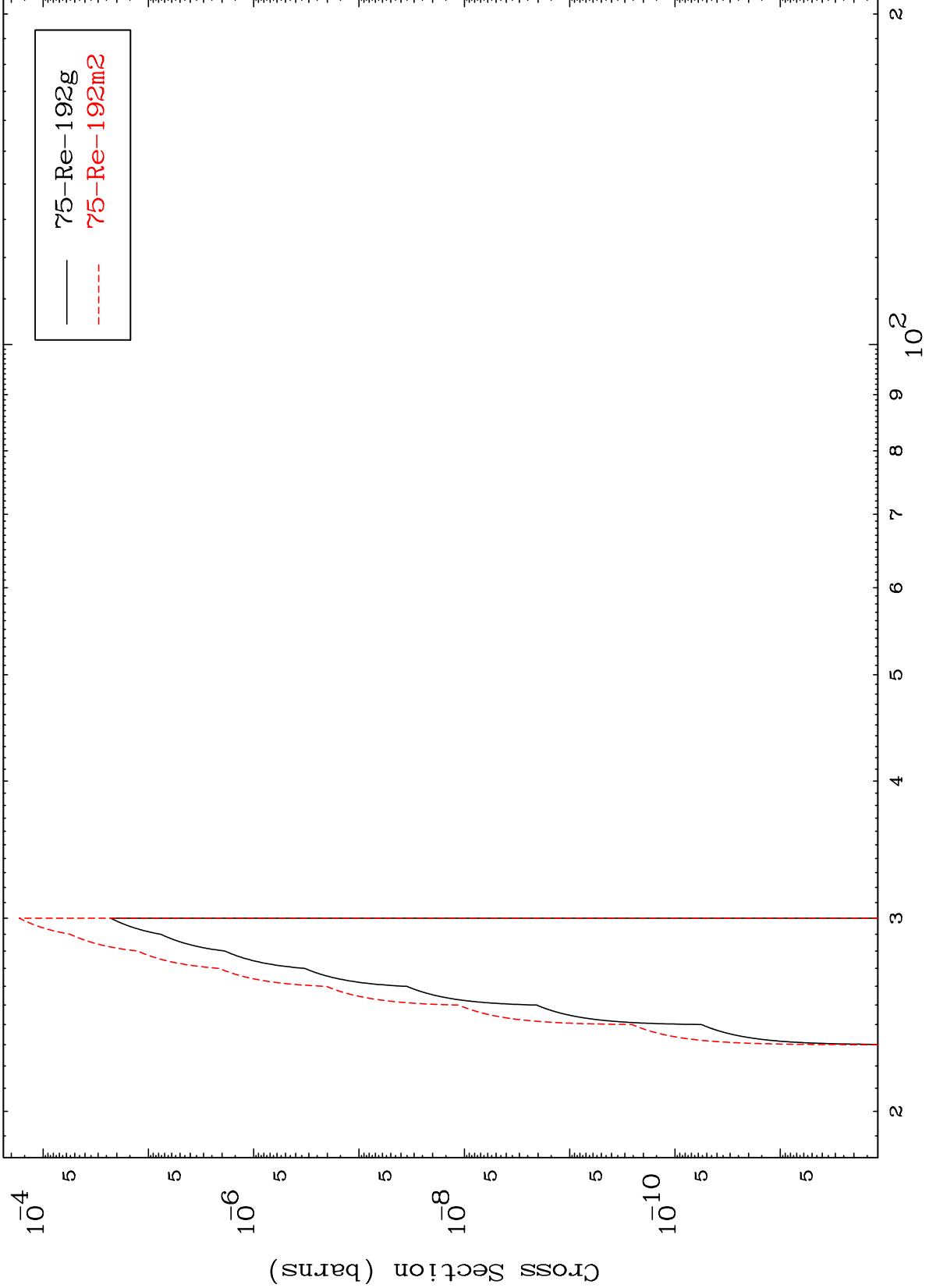


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(n,2n) d

76-0s-195m

Radionuclide Production Cross Section



20

Incident Energy (MeV)

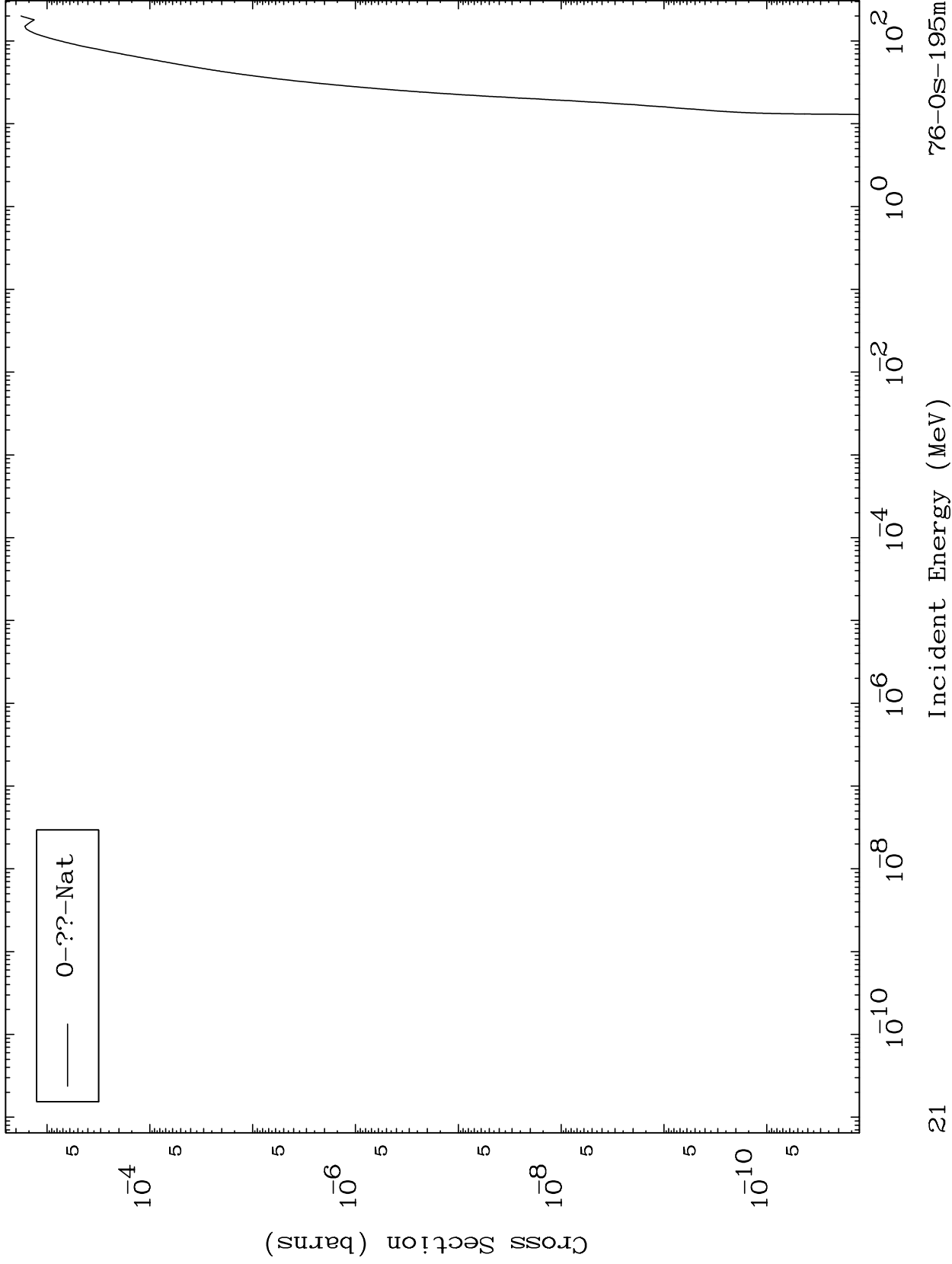
76-0s-195m

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Fission

76-0s-195m

Radionuclide Production Cross Section

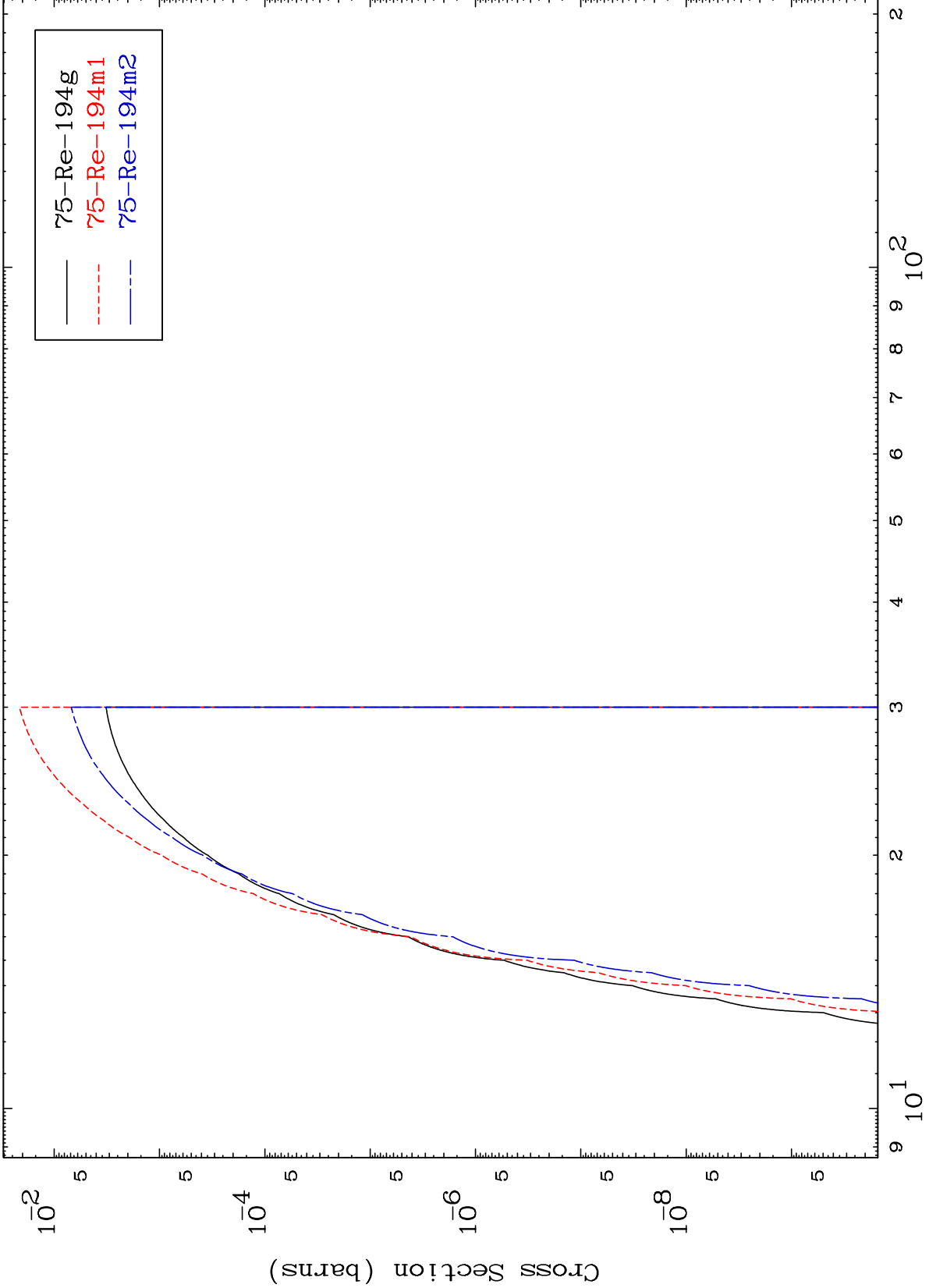


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(n,n') p

76-0s-195m

Radionuclide Production Cross Section



22

Incident Energy (MeV)

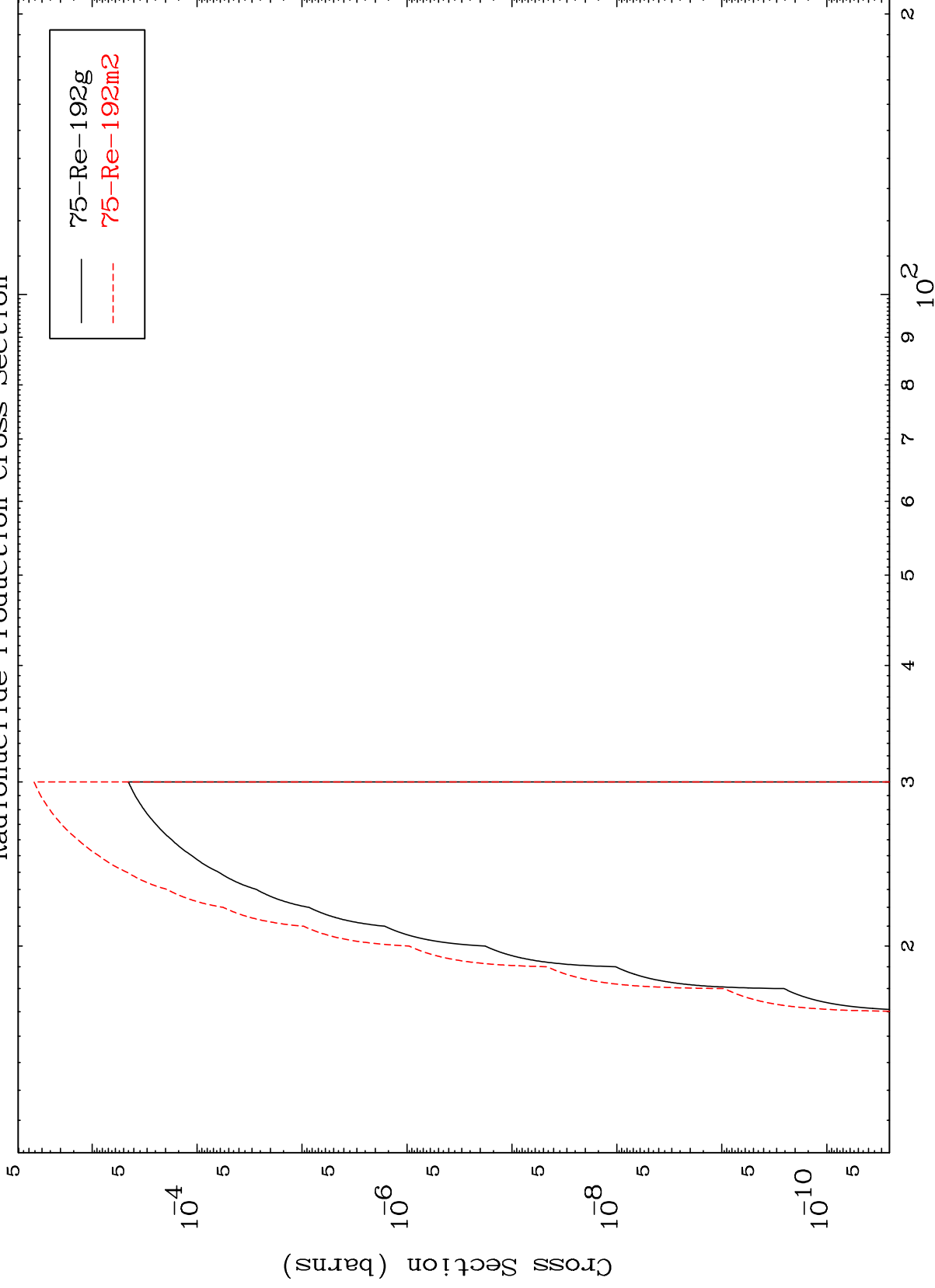
76-0s-195m

MAT 7659

(n,n') t

76-0s-195m

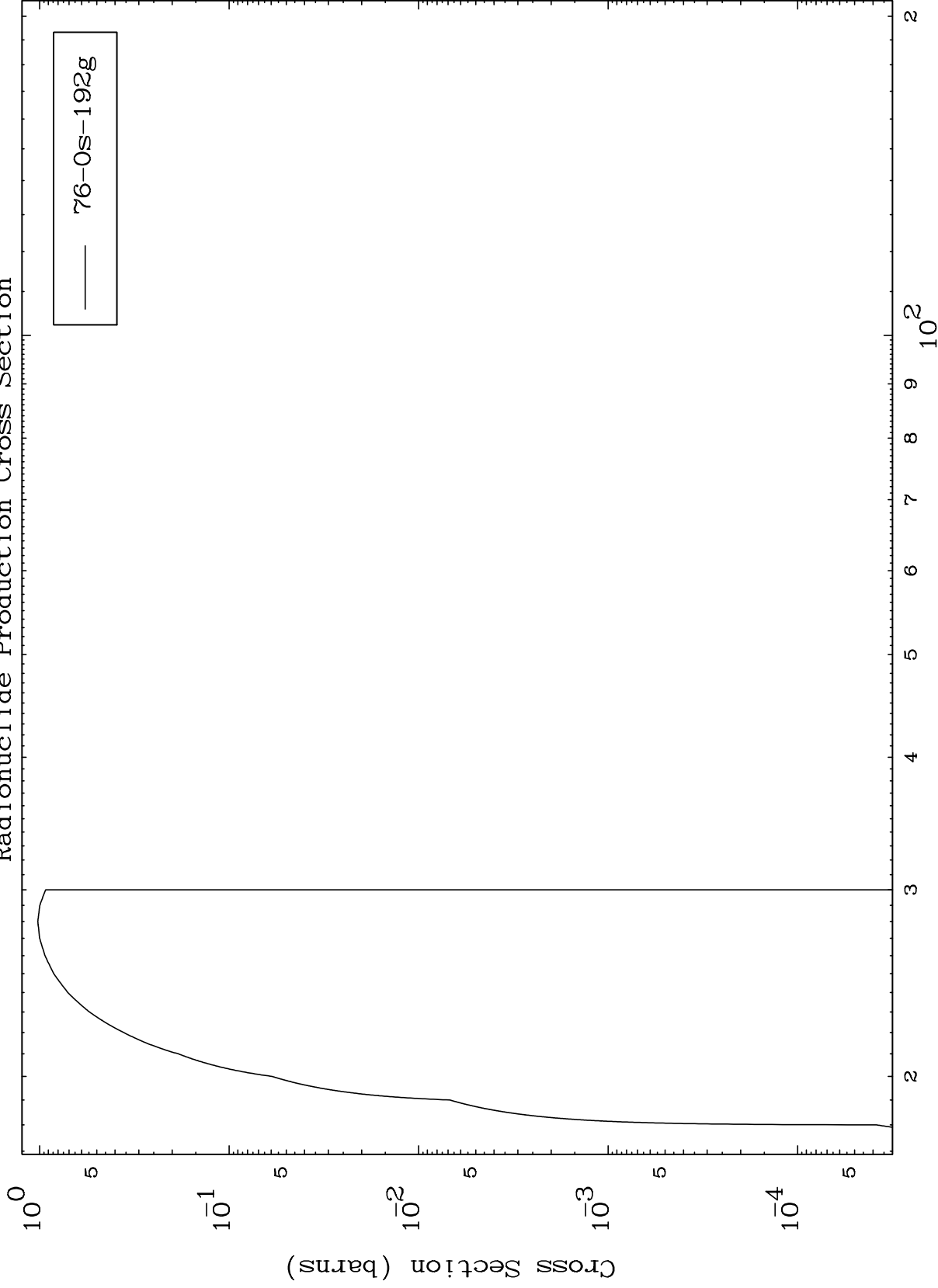
Radionuclide Production Cross Section



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76-0s-195m

(n,4n)
Radionuclide Production Cross Section



24

Incident Energy (MeV)

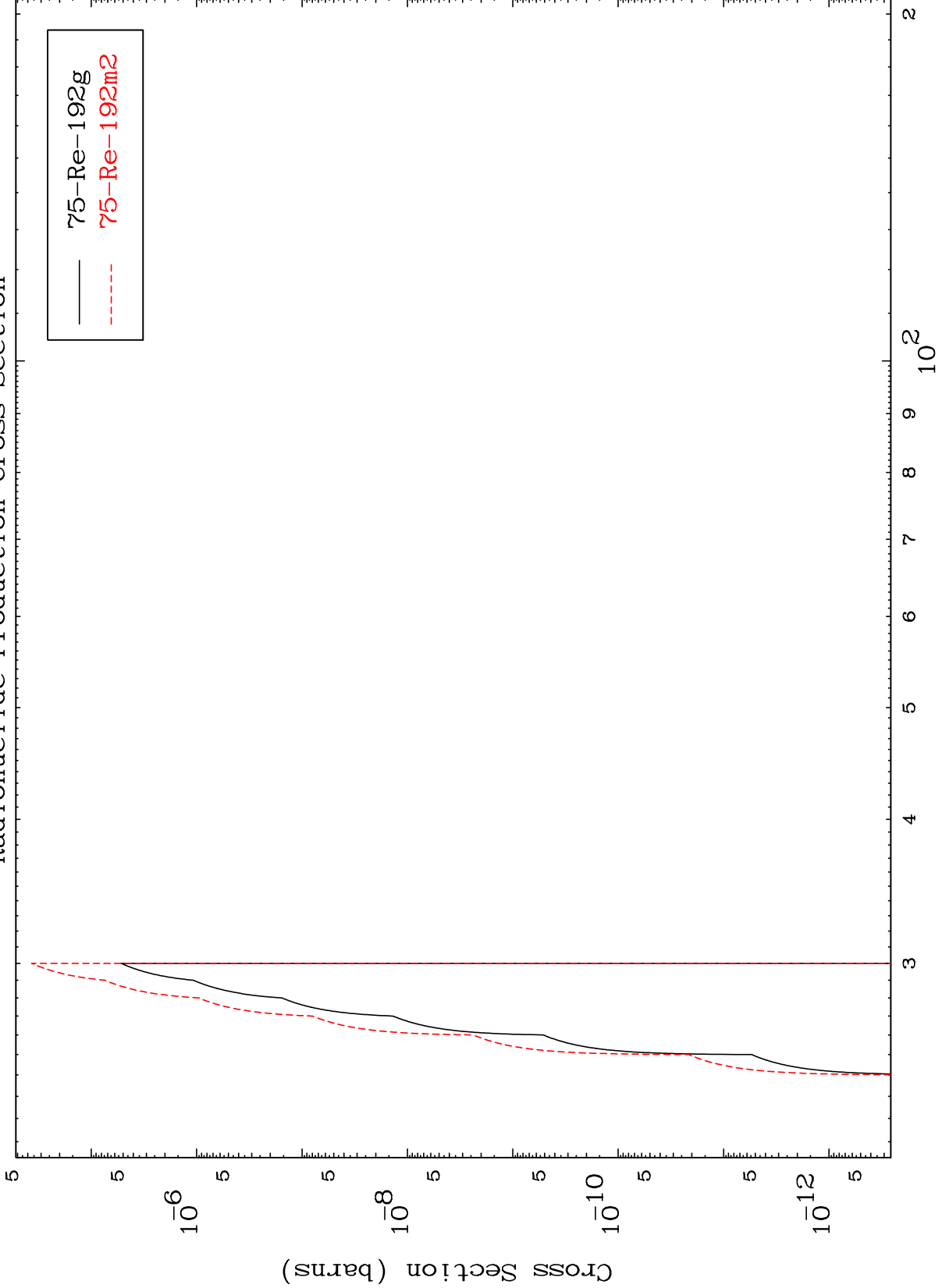
76-0s-195m

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(n,3n) p

76-0s-195m

Radionuclide Production Cross Section



75-Re-192g
75-Re-192m2

25

Incident Energy (MeV)

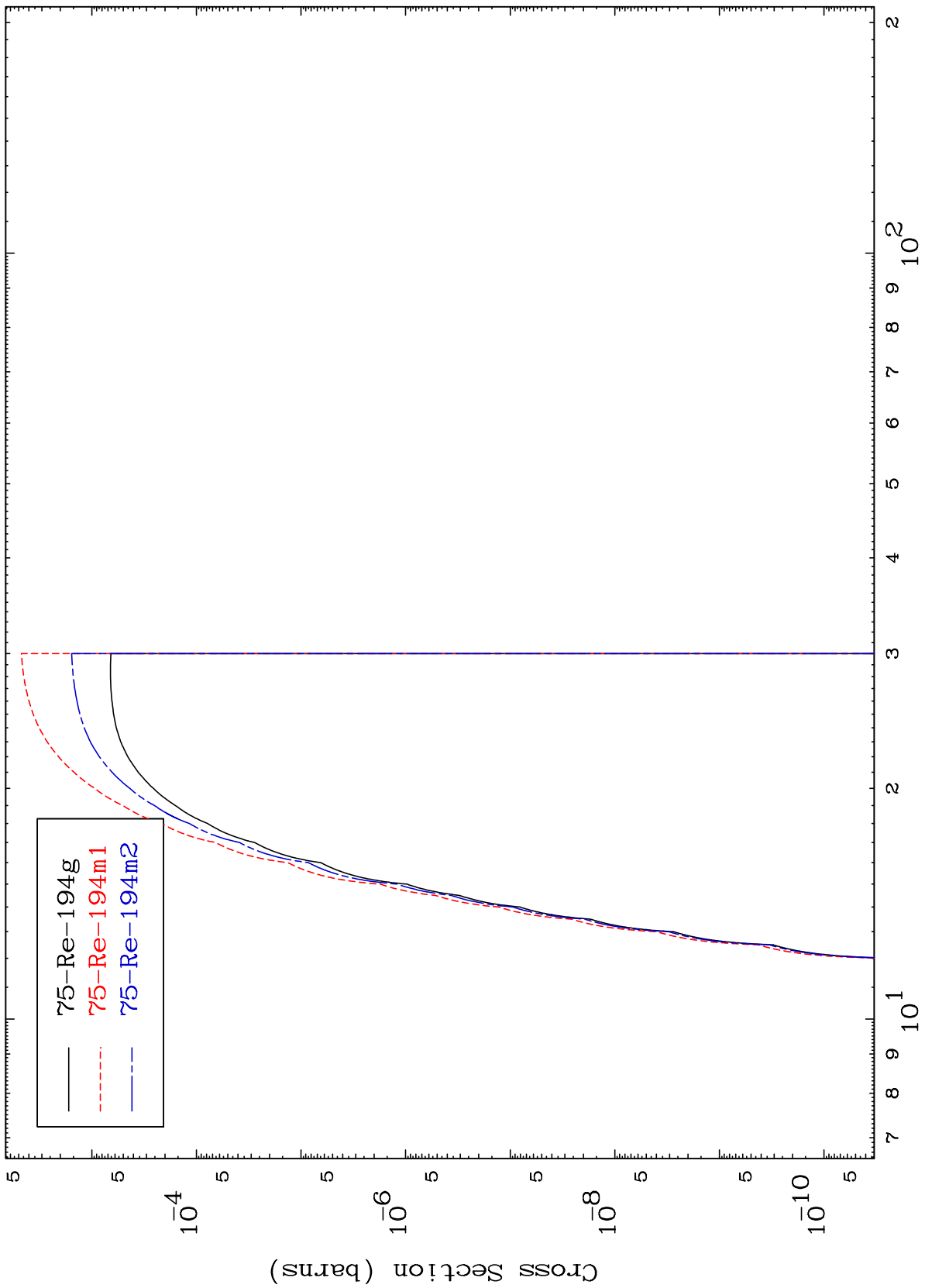
76-0s-195m

MAT 7659

(n,d)

76-0s-195m

Radionuclide Production Cross Section



26

Incident Energy (MeV)

76-0s-195m