

Program EVALPLOT  
(Version 2021-1)

by

Dermott E. Cullen  
(Present Contact Information)

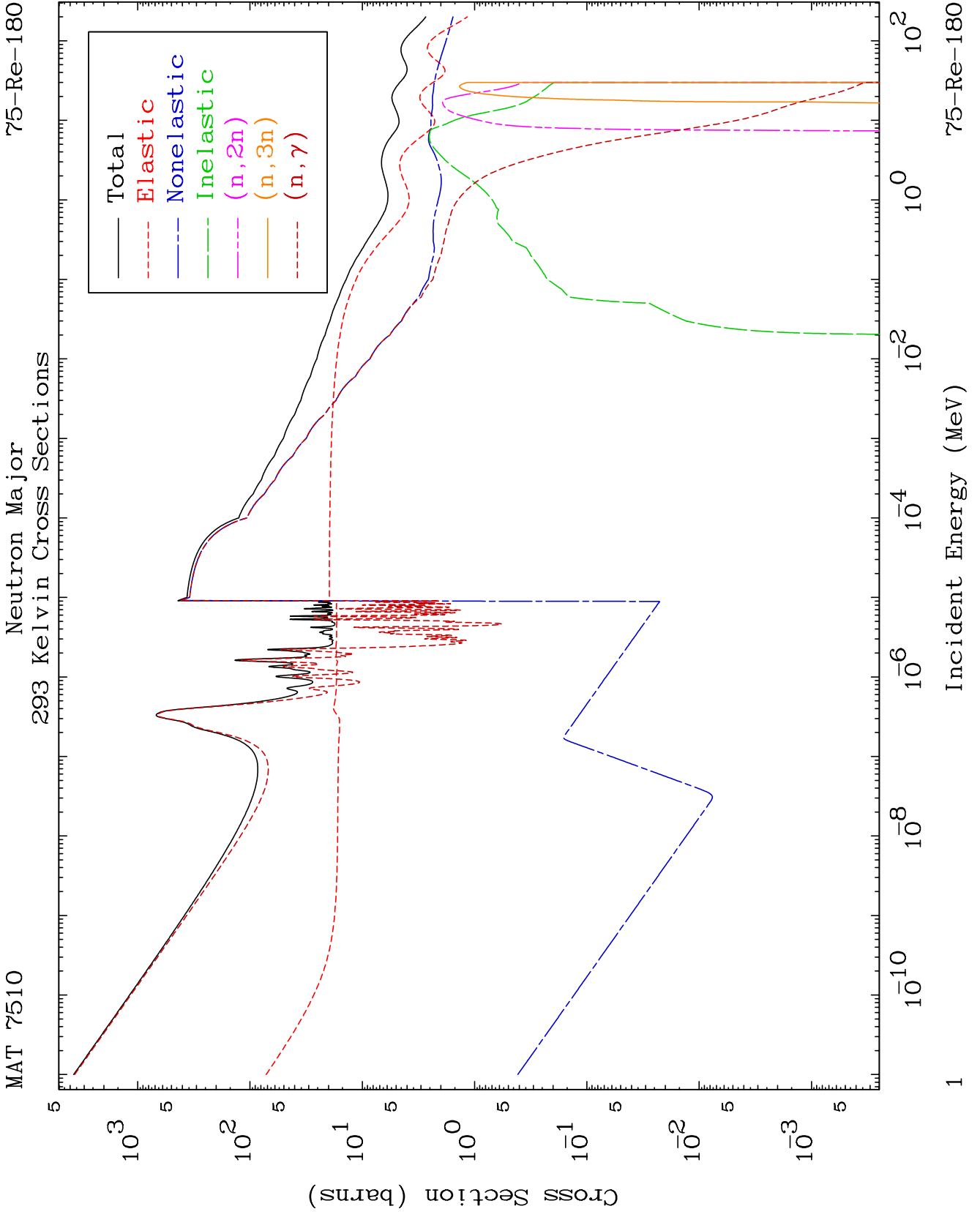
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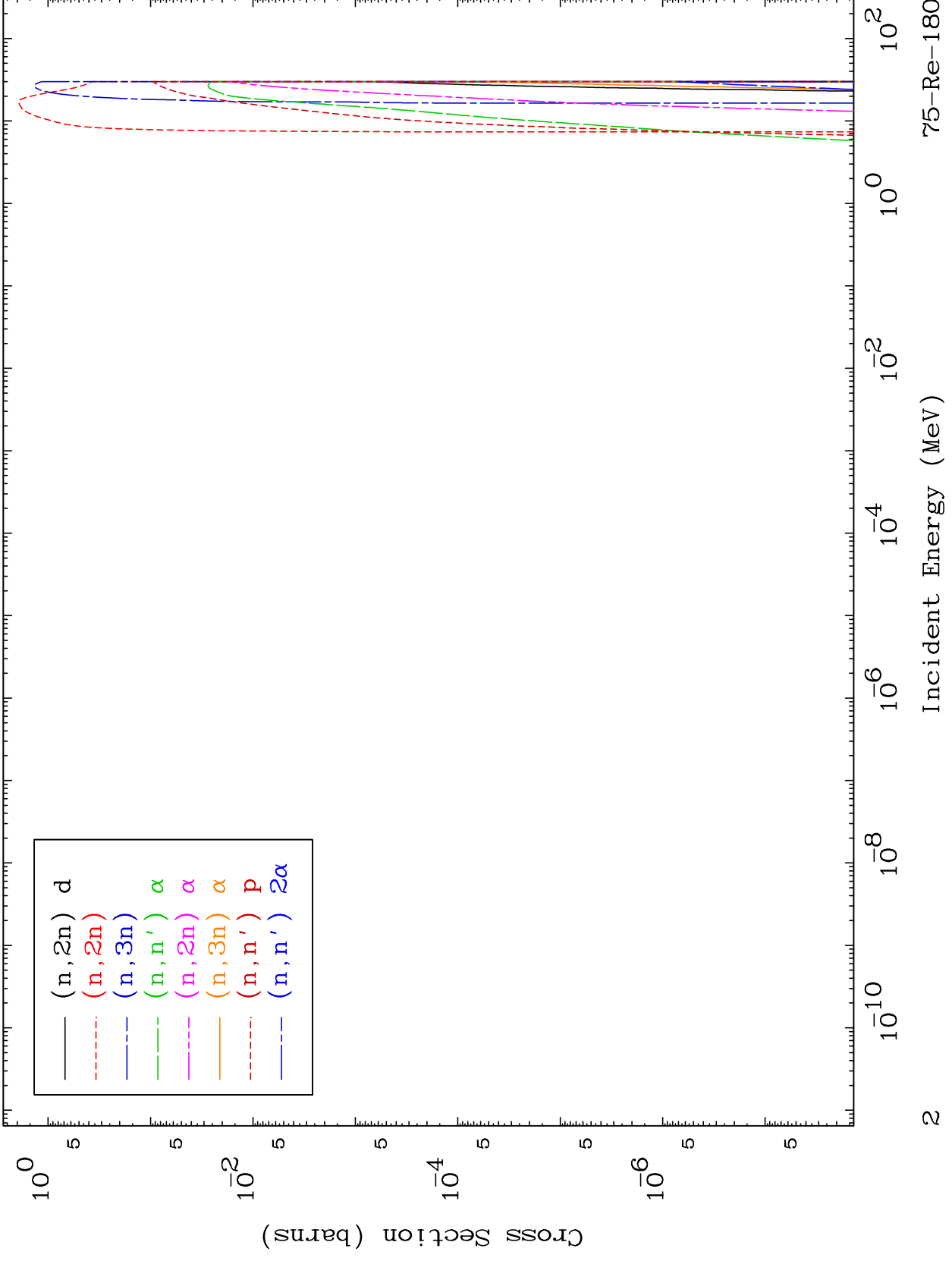
Press Mouse Button to Start



MAT 7510

Neutron Absorption  
293 Kelvin Cross Sections

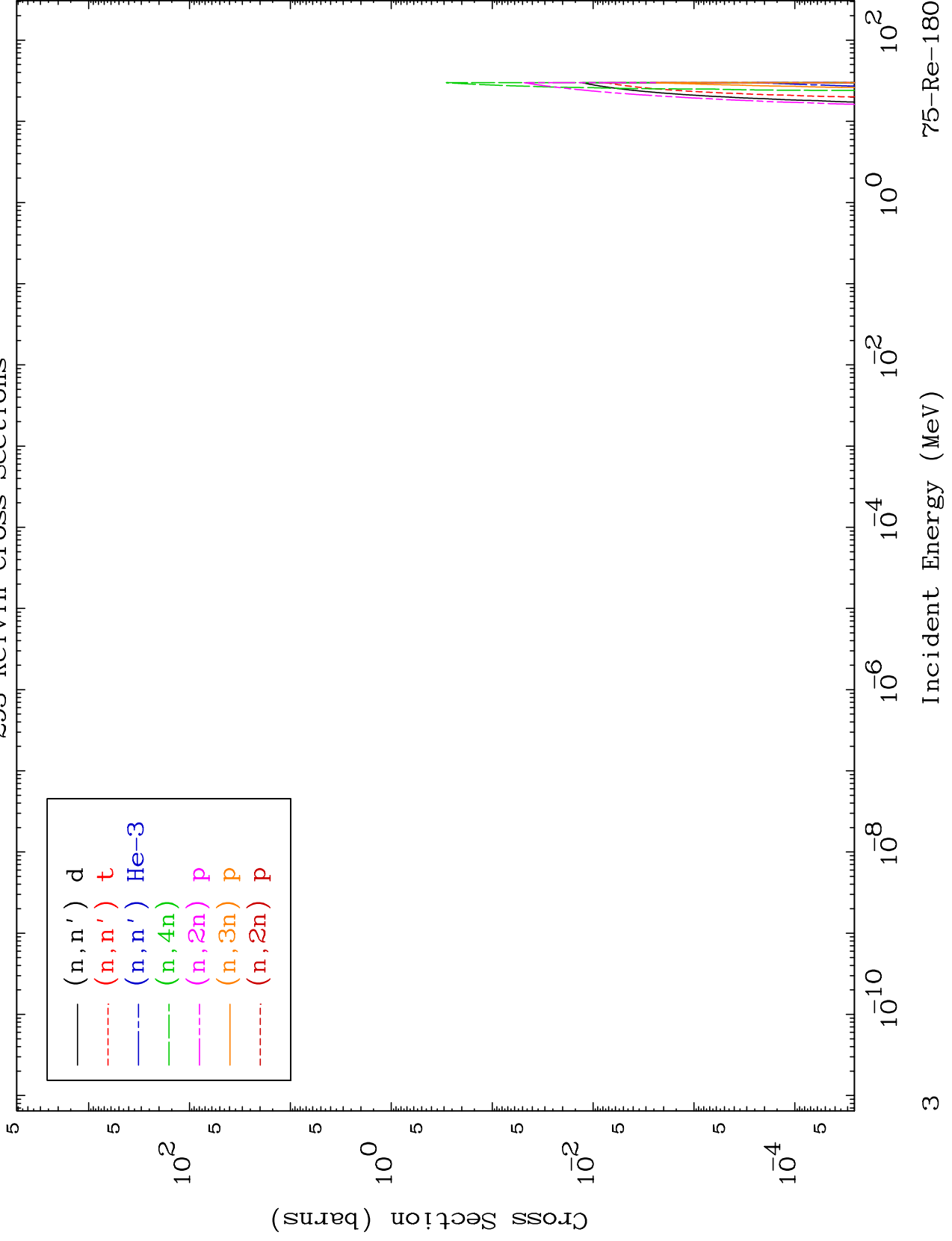
75-Re-180

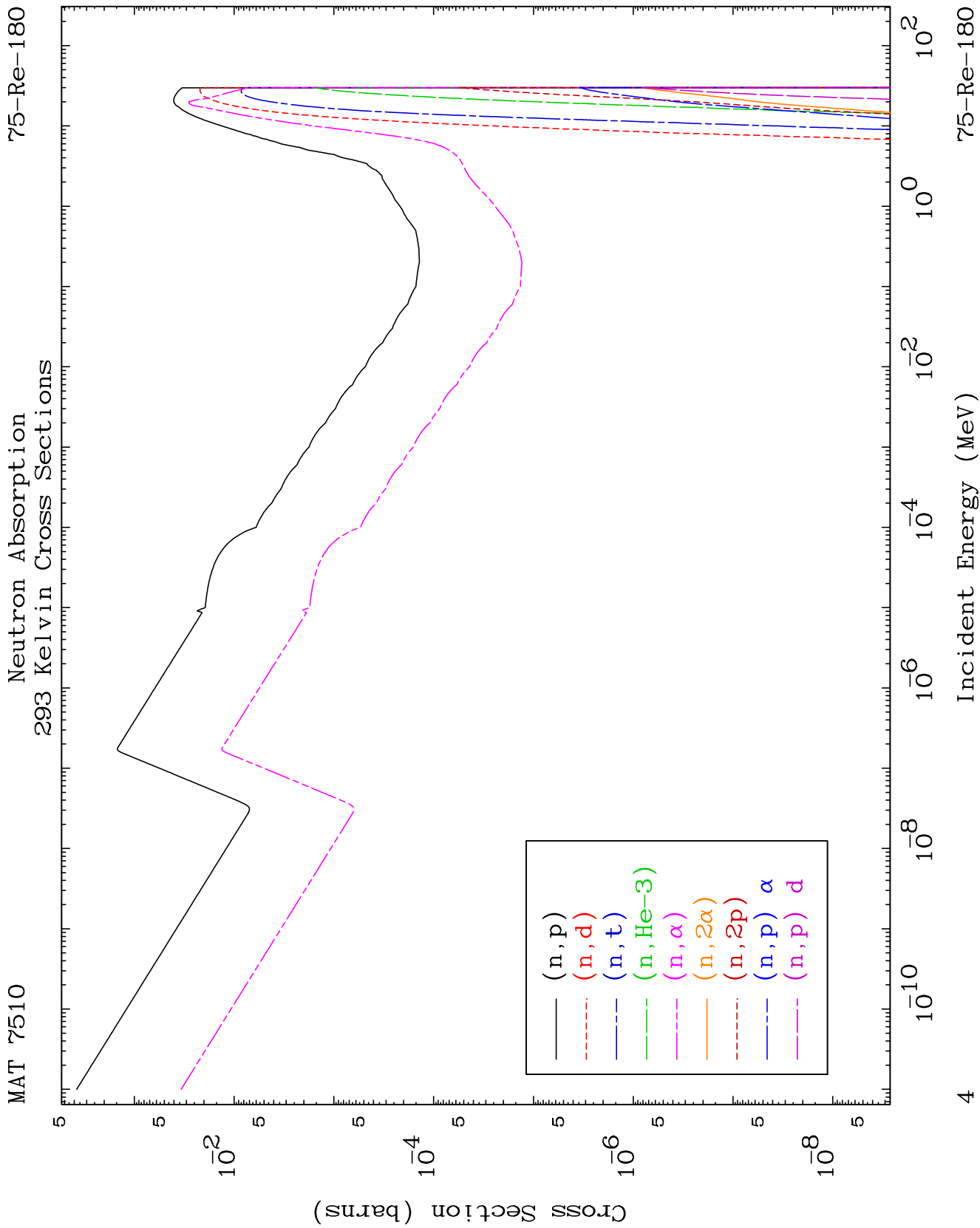


MAT 7510

Neutron Absorption  
293 Kelvin Cross Sections

75-Re-180

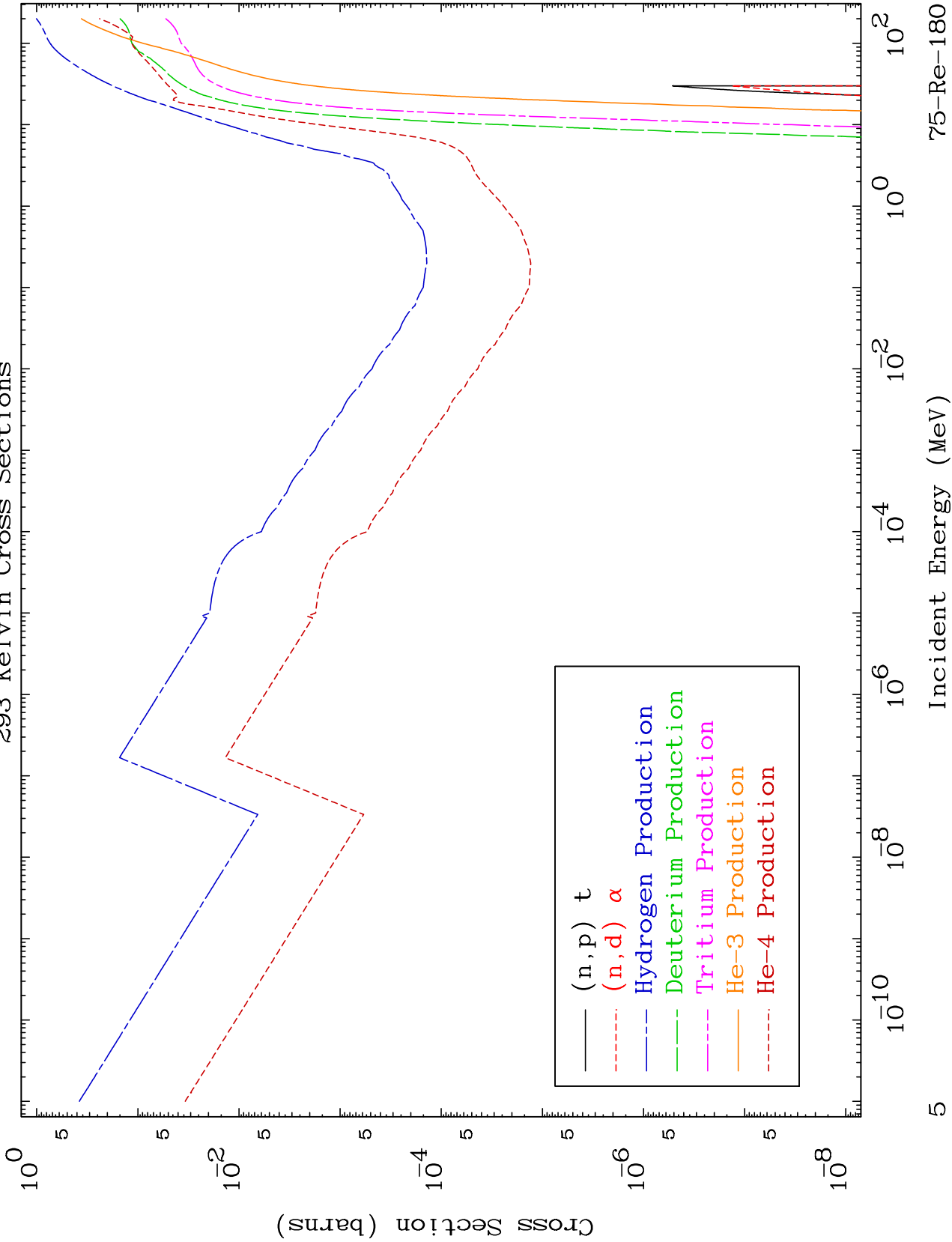




MAT 7510

Neutron Absorption  
293 Kelvin Cross Sections

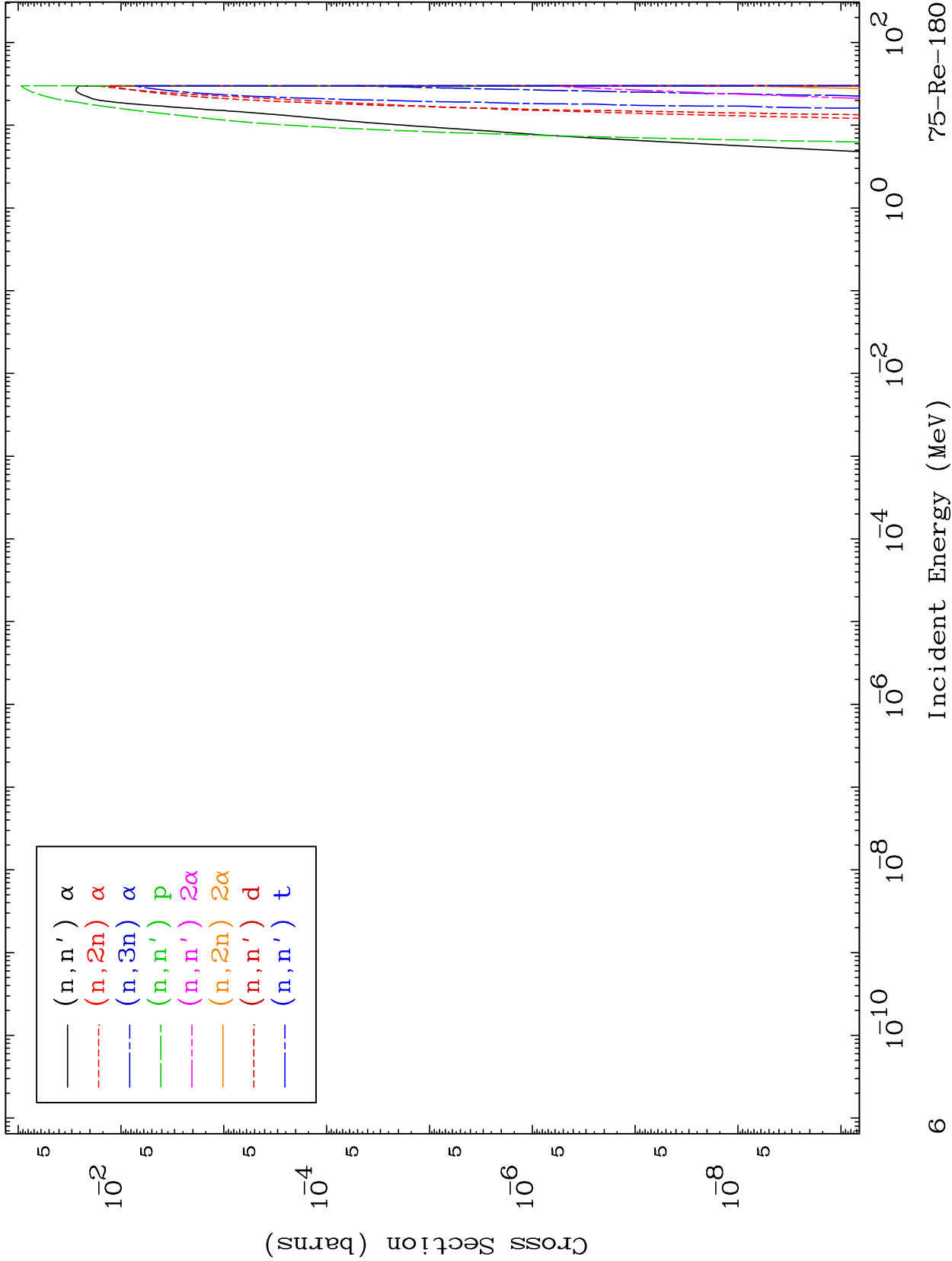
75-Re-180

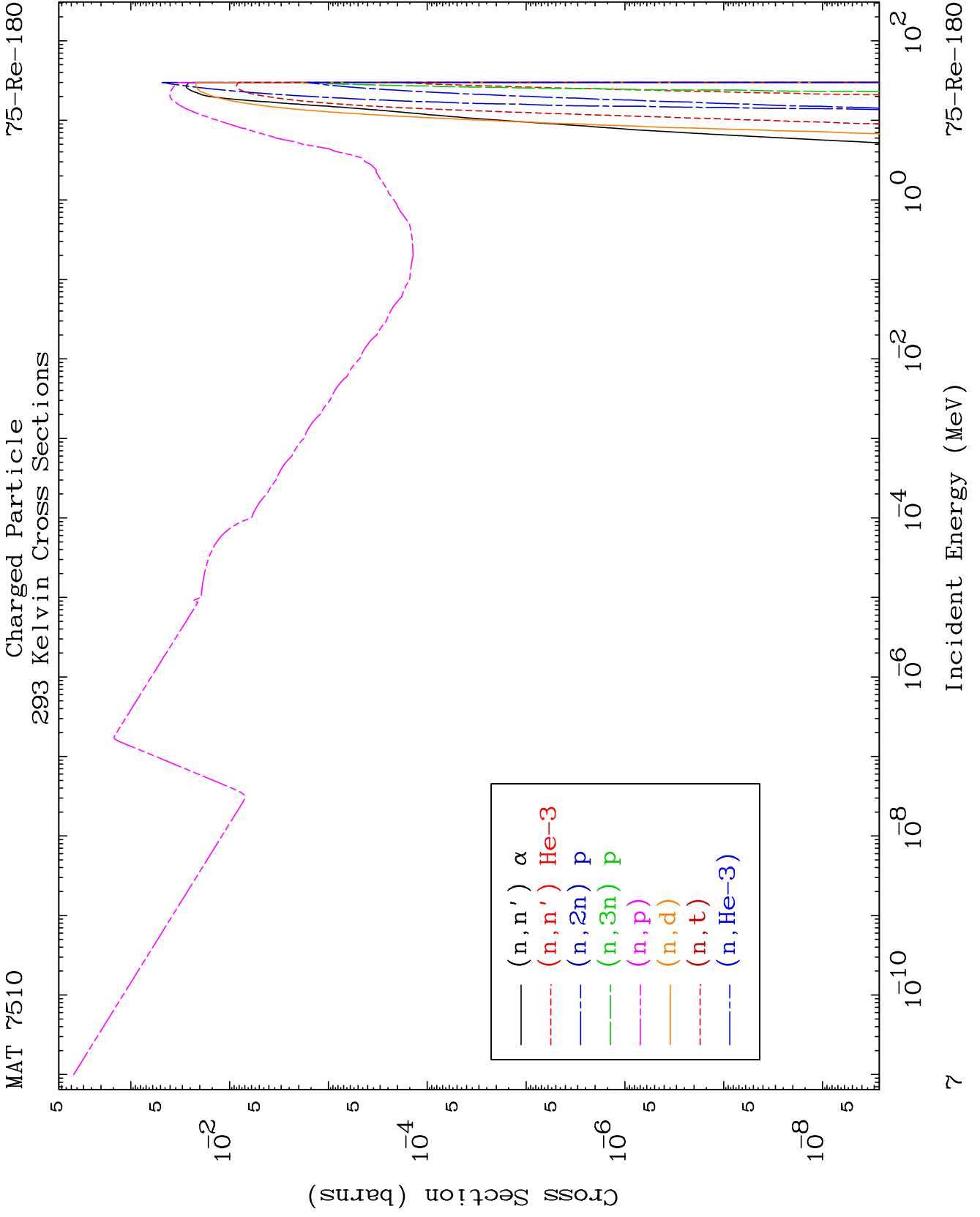


MAT 7510

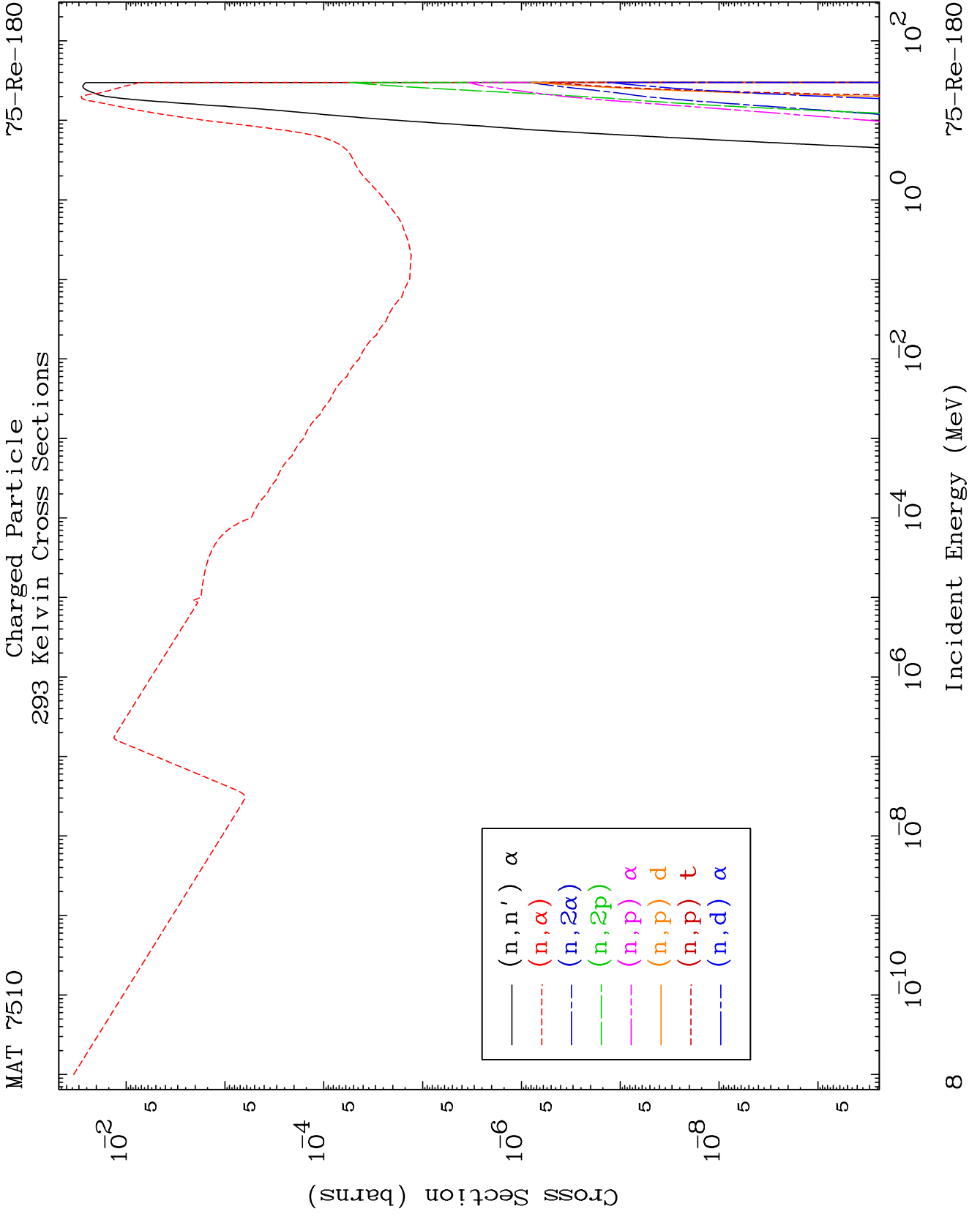
Charged Particle  
293 Kelvin Cross Sections

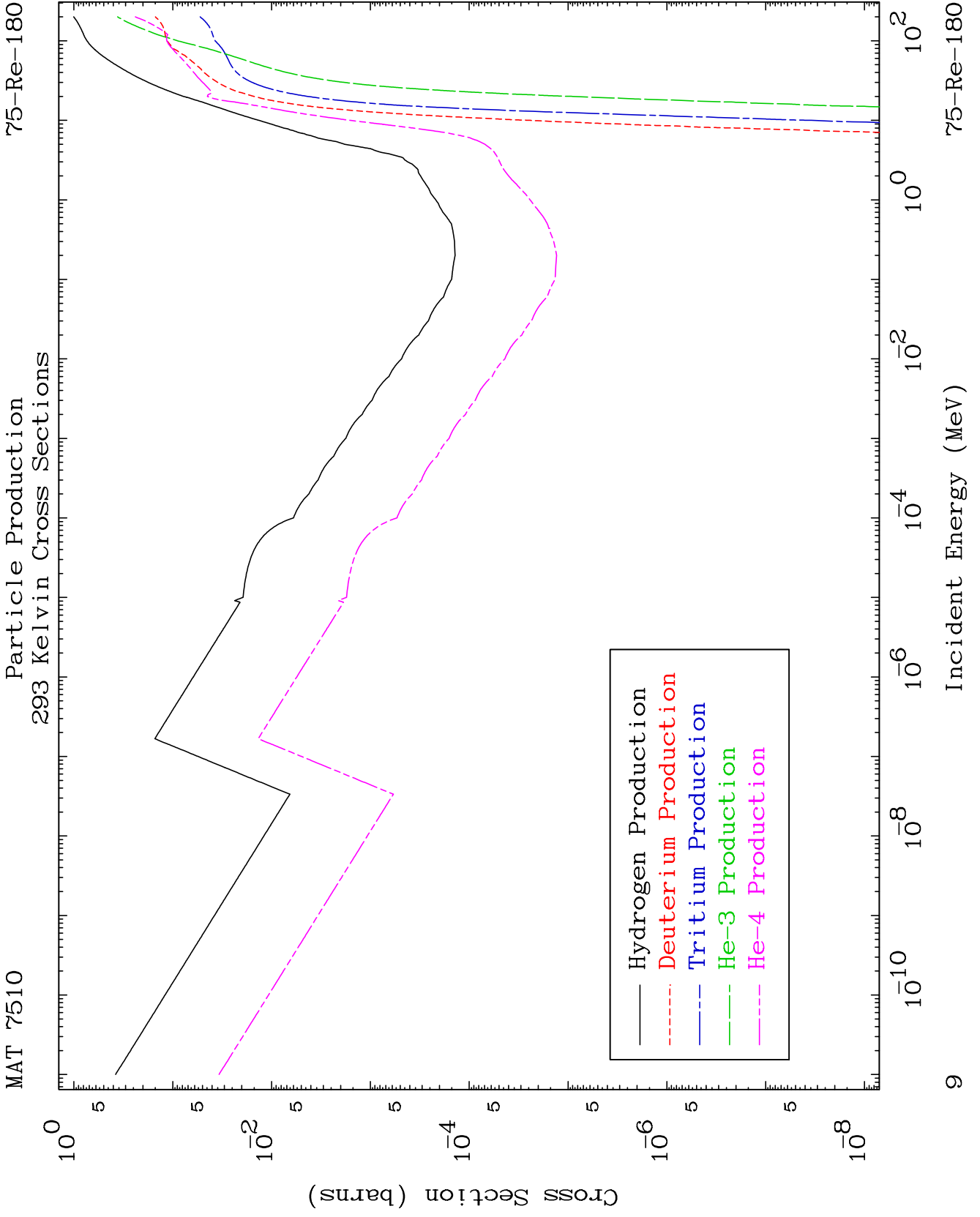
75-Re-180







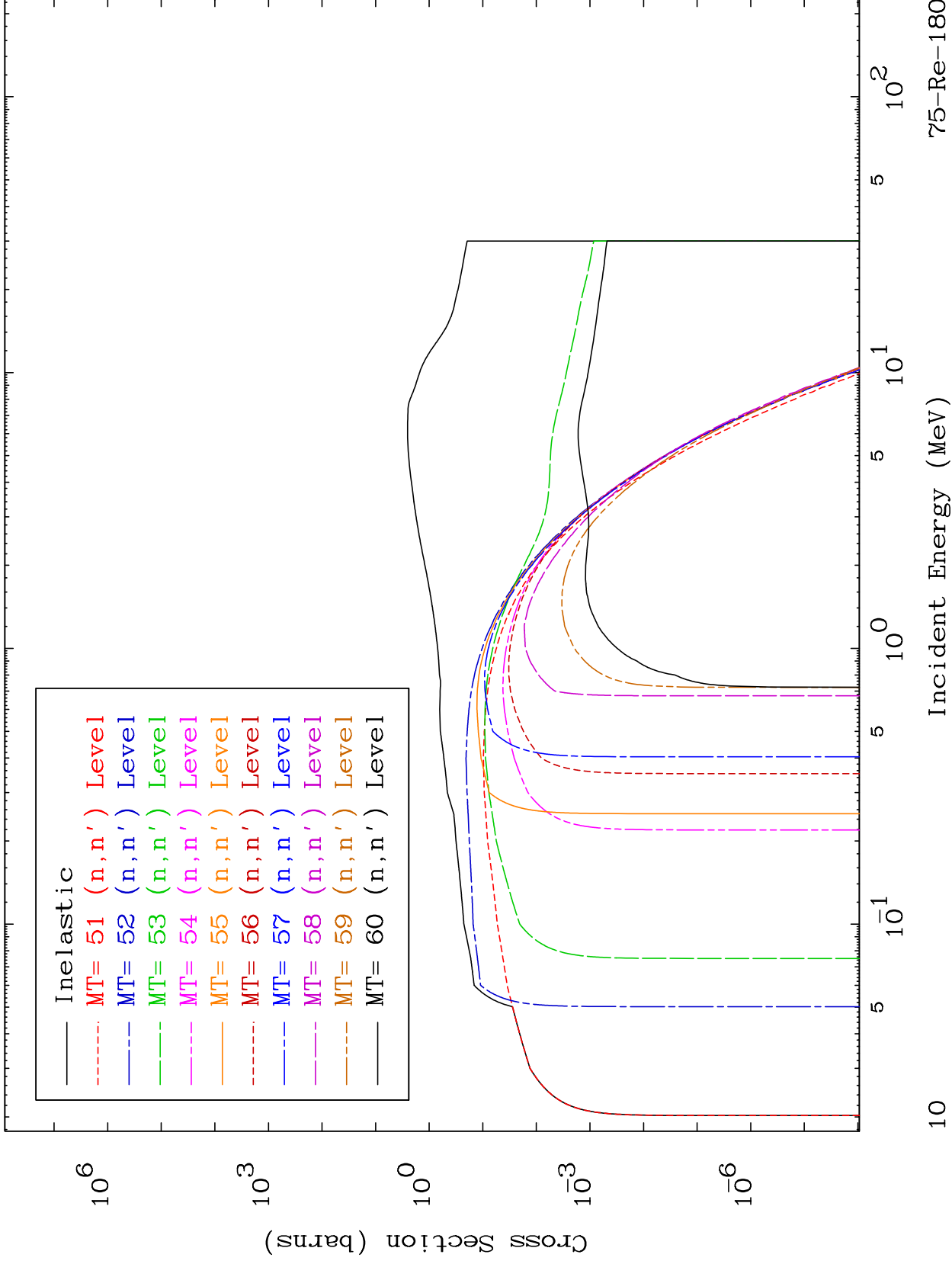




MAT 7510

(n,n') Levels  
293 Kelvin Cross Sections

75-Re-180



10

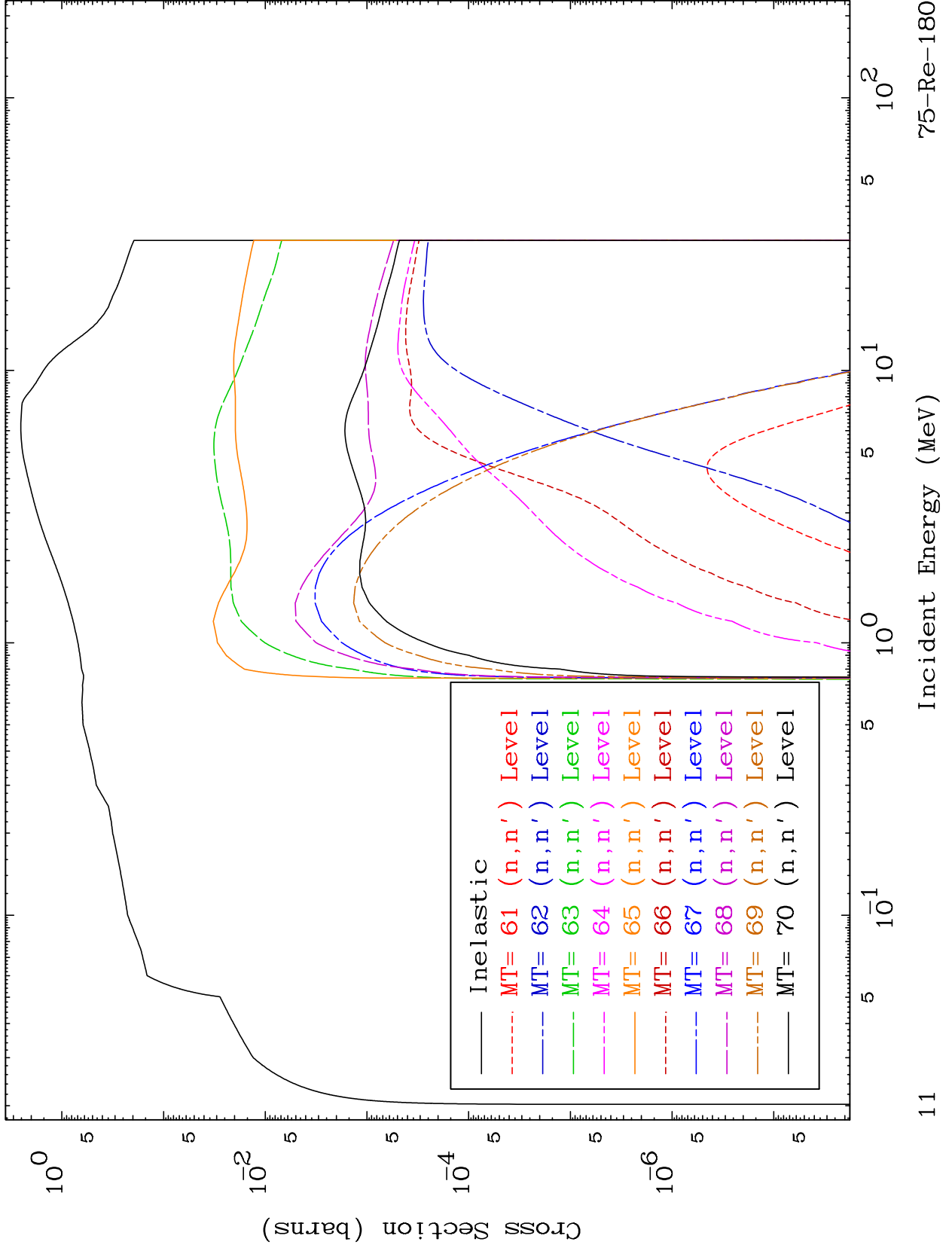
Incident Energy (MeV)

75-Re-180

MAT 7510

(n,n') Levels  
293 Kelvin Cross Sections

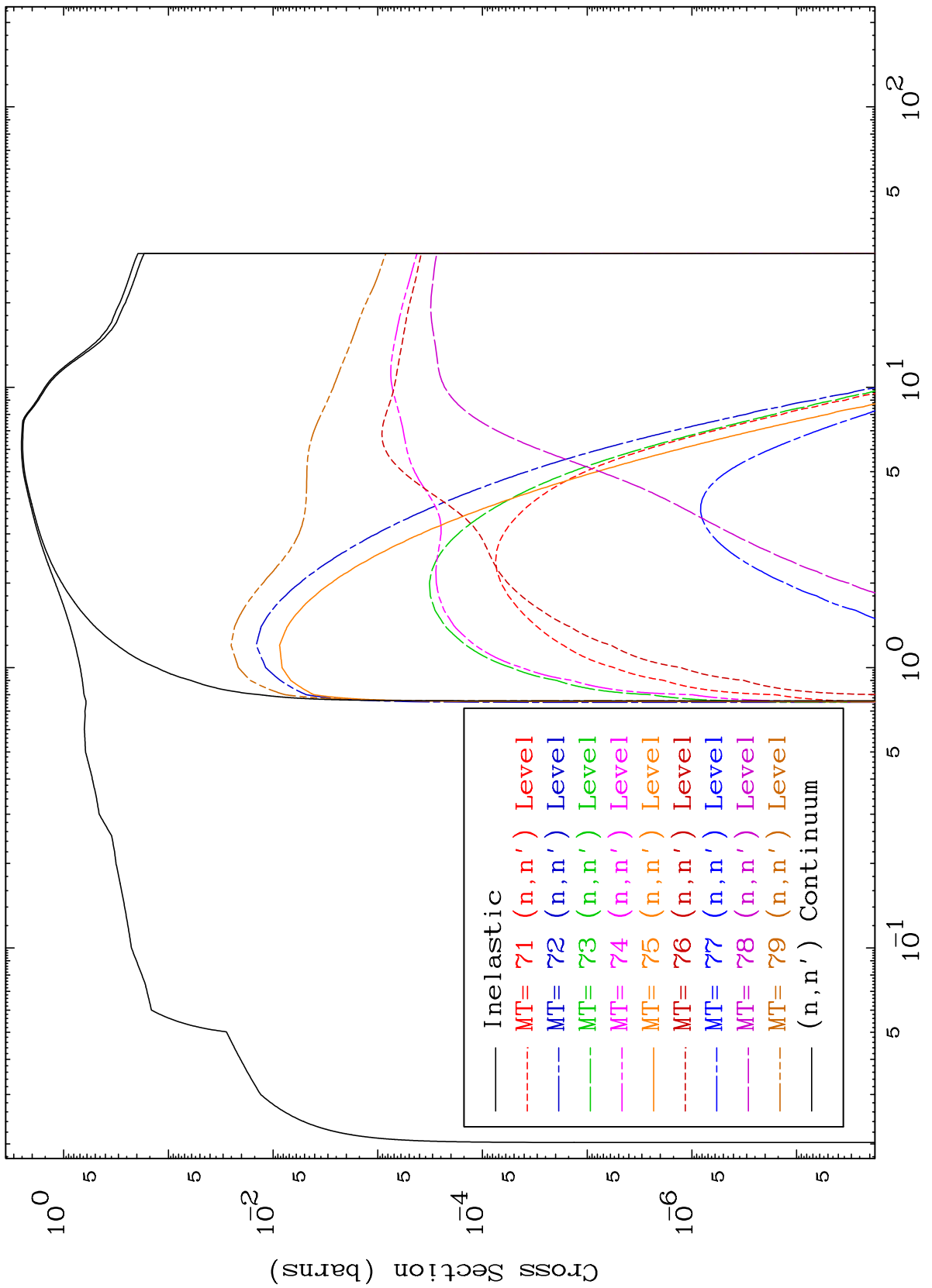
75-Re-180

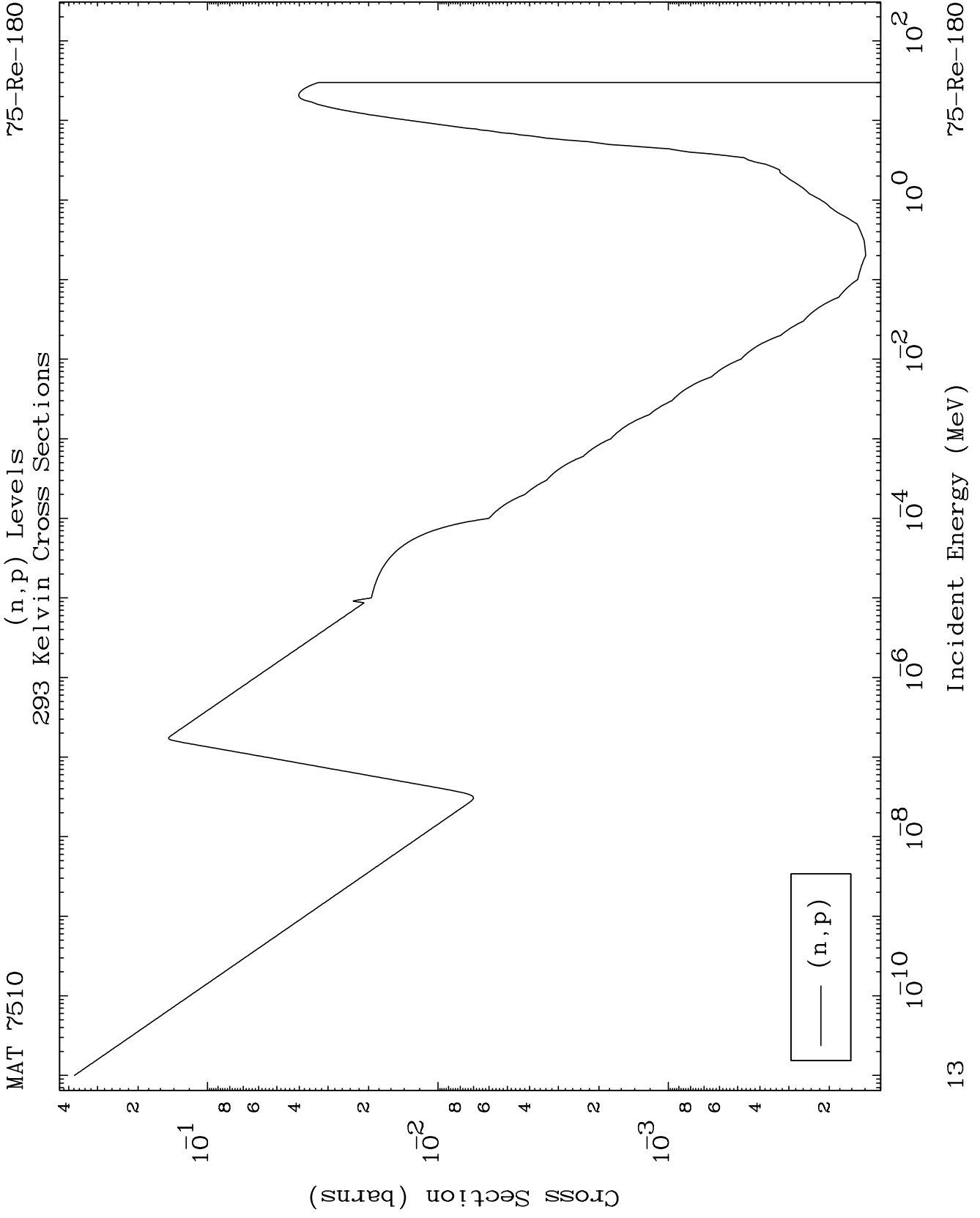


MAT 7510

(n,n') Levels  
293 Kelvin Cross Sections

75-Re-180

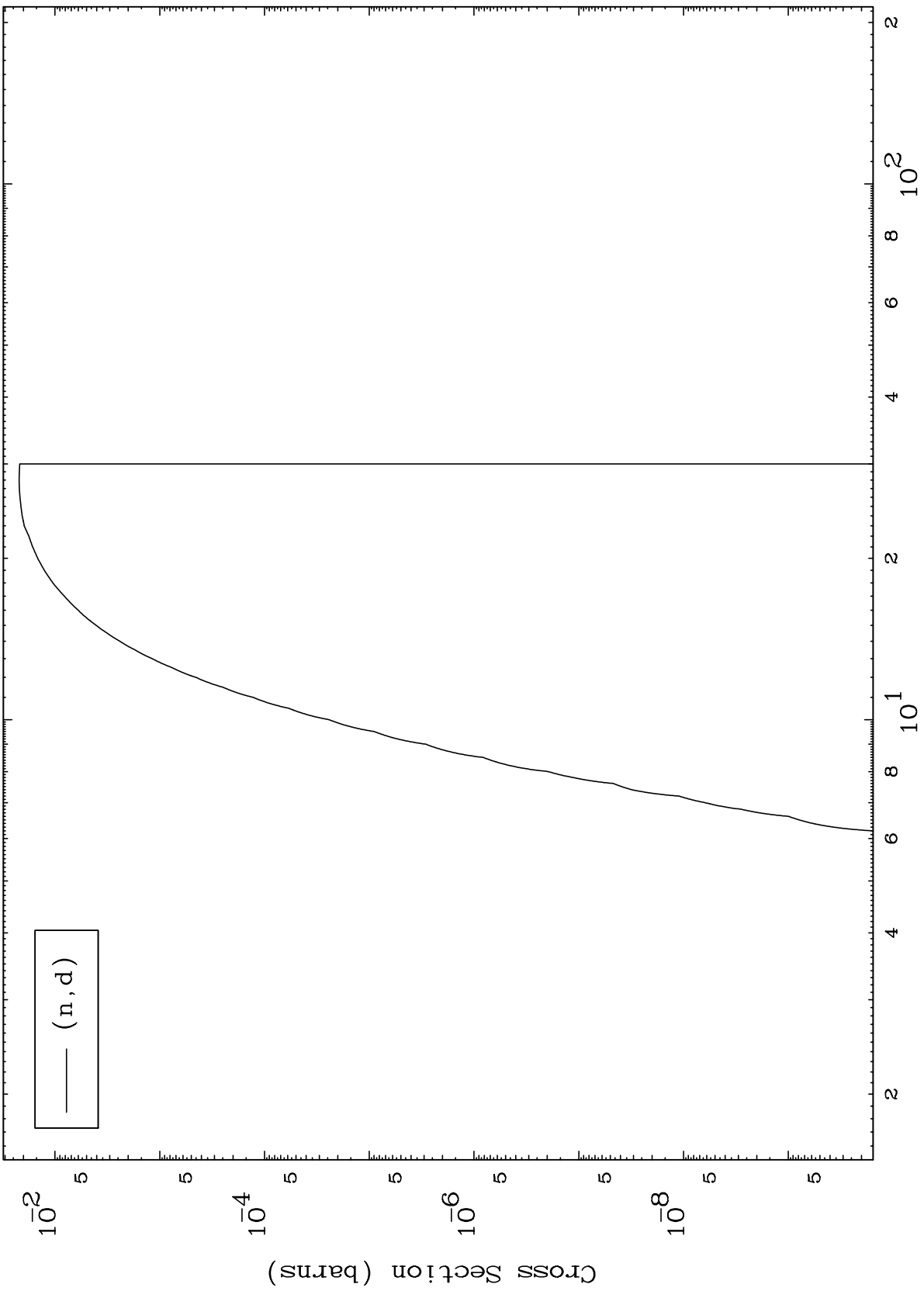




MAT 7510

(n,d) Levels  
293 Kelvin Cross Sections

75-Re-180

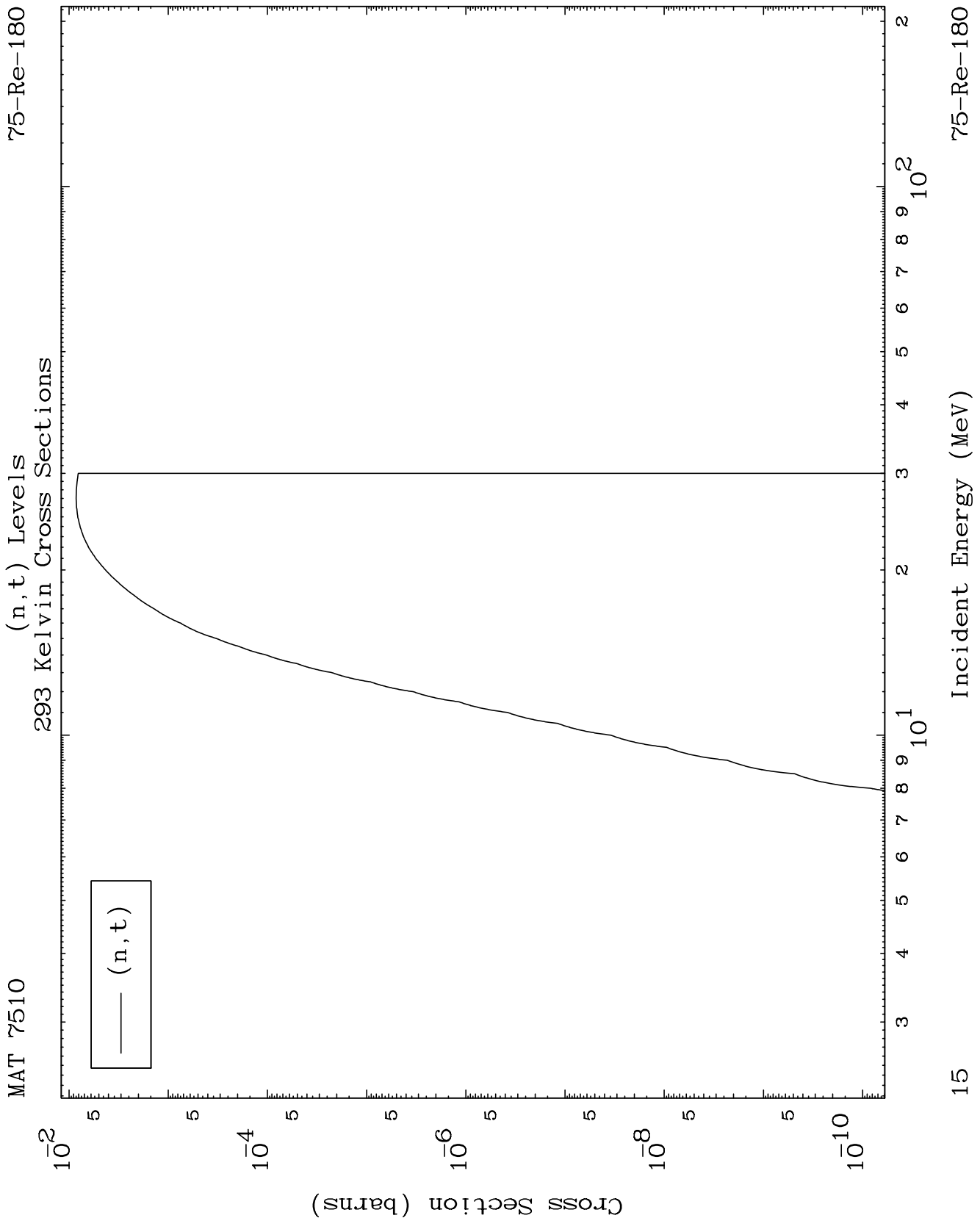


(n,d)

75-Re-180

Incident Energy (MeV)

14

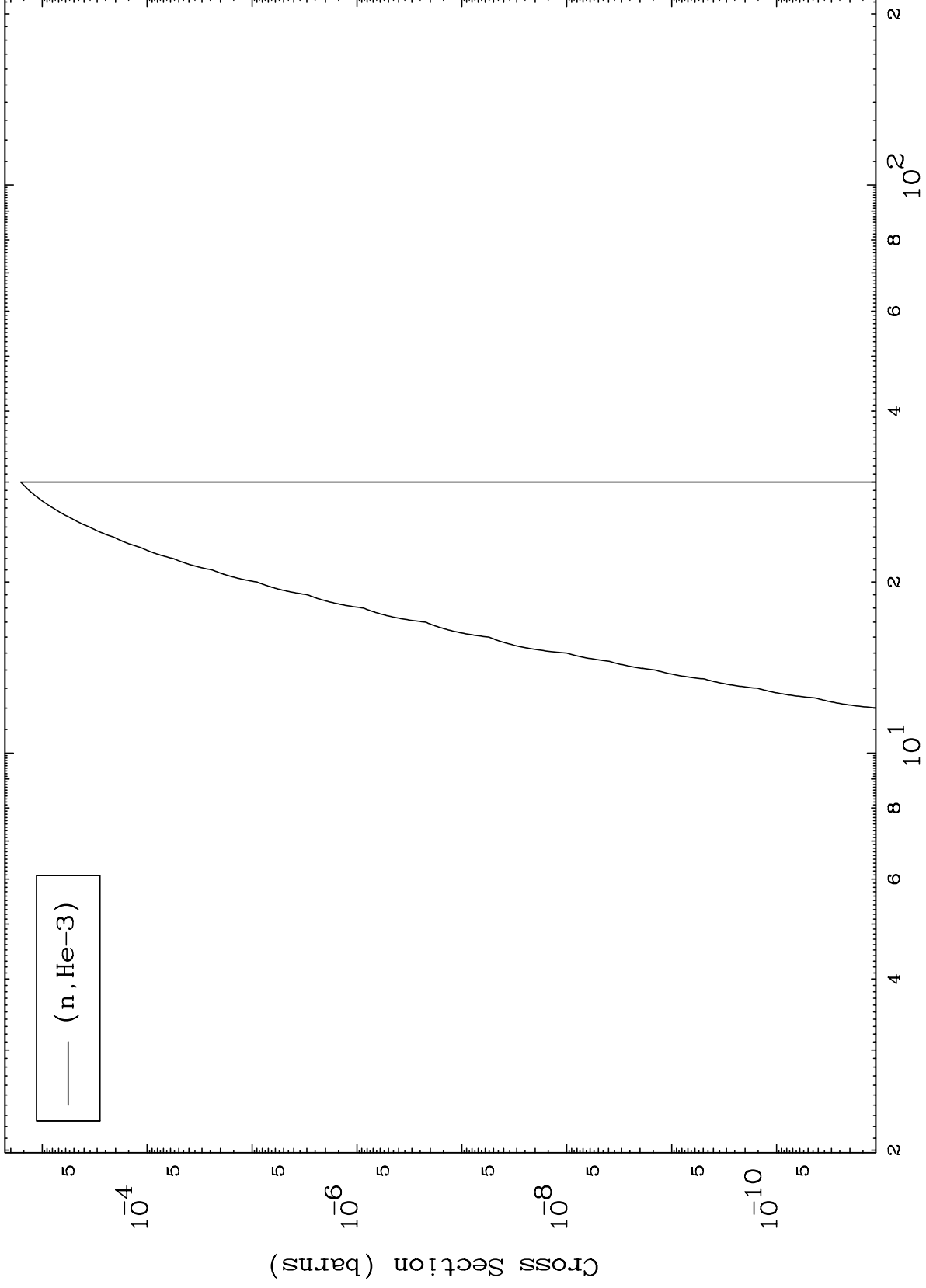




MAT 7510

(n,He3) Levels  
293 Kelvin Cross Sections

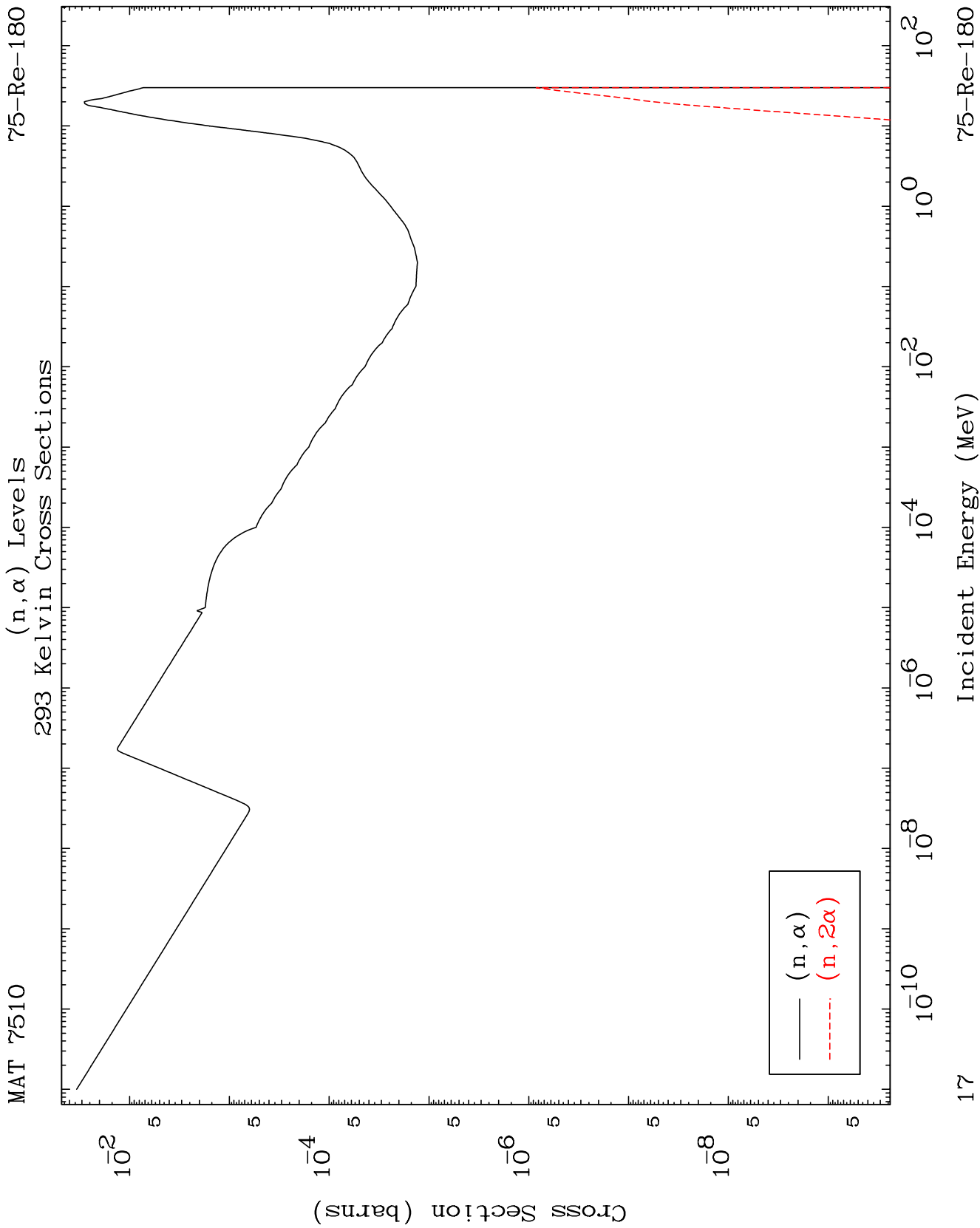
75-Re-180



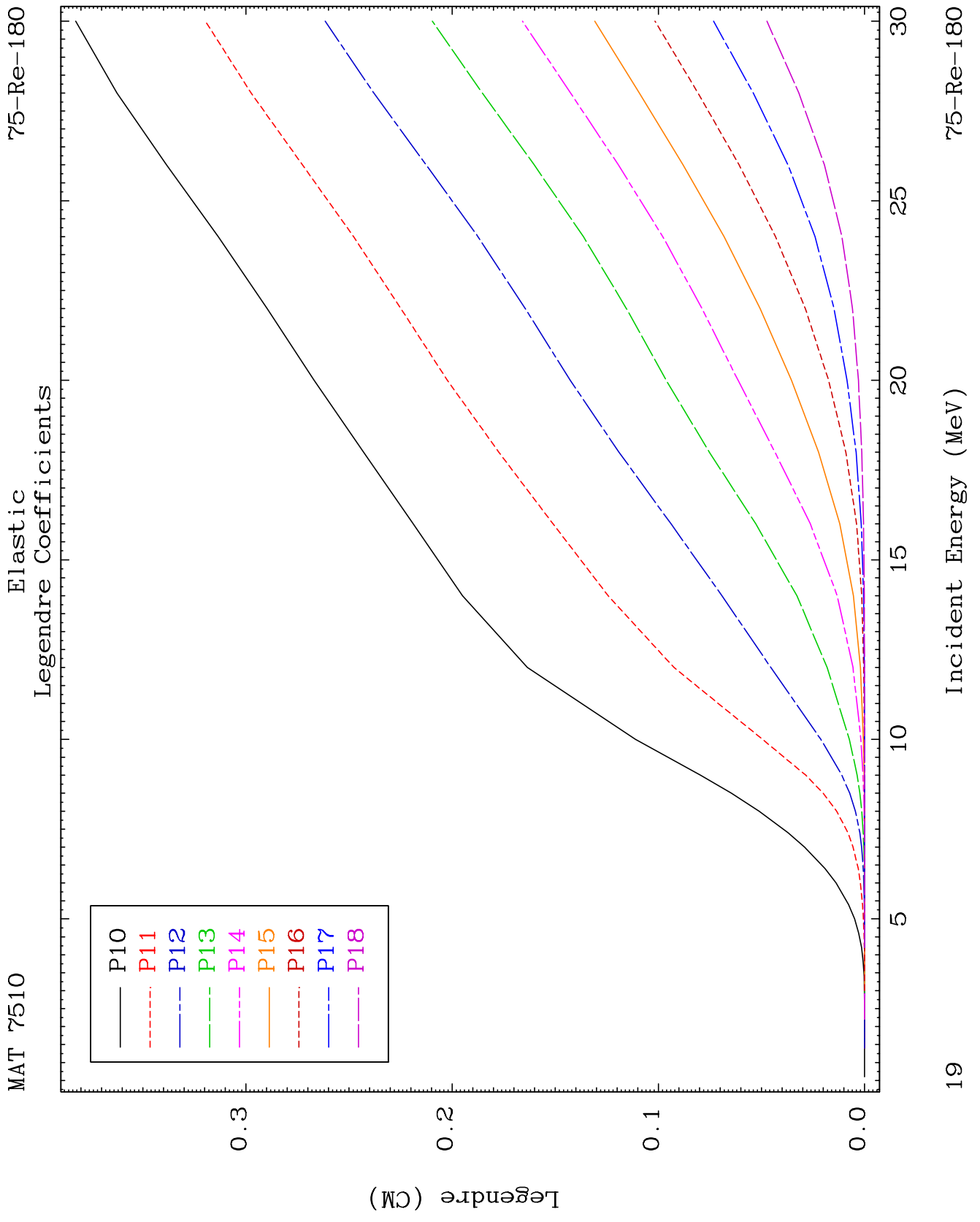
16

Incident Energy (MeV)

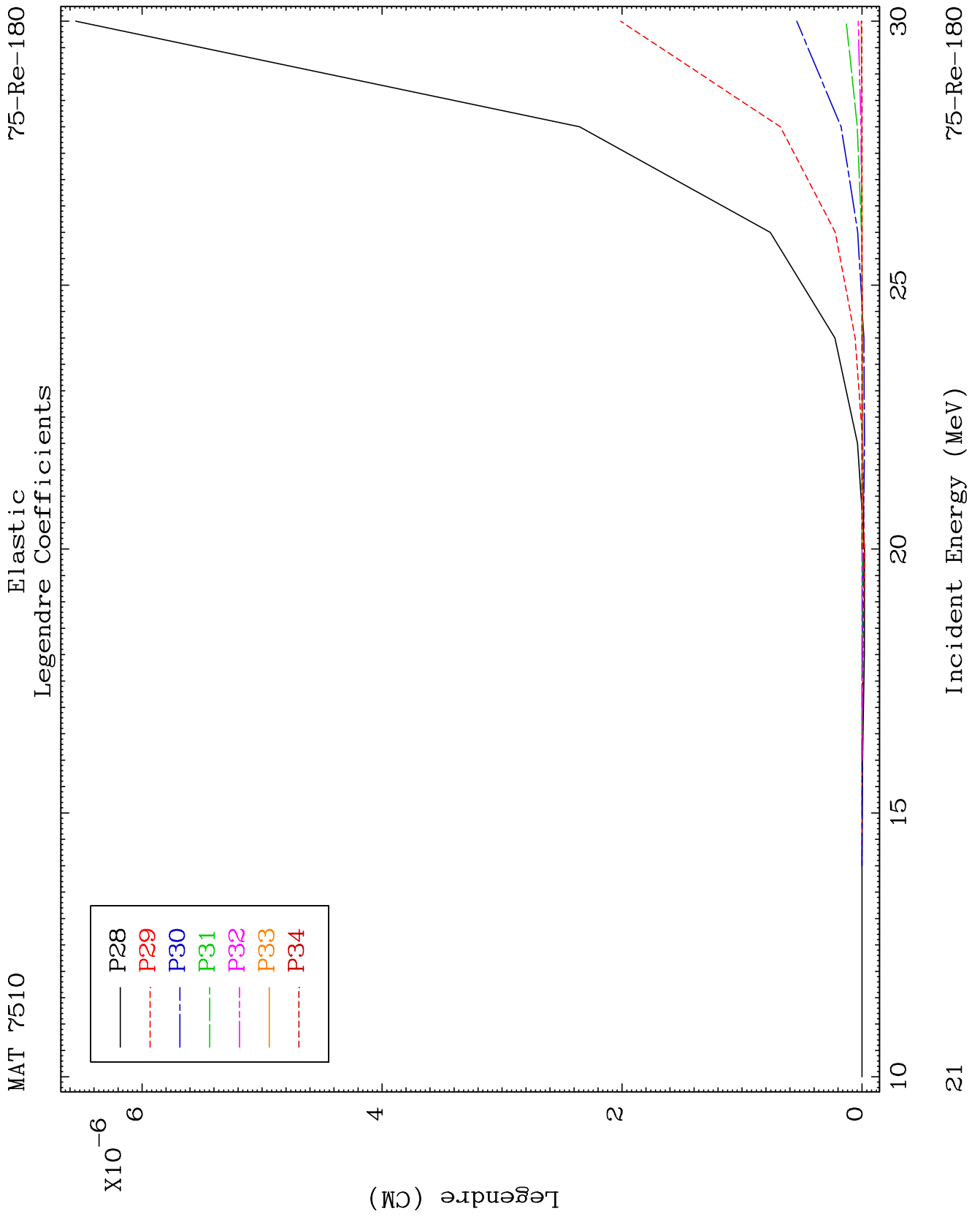
75-Re-180

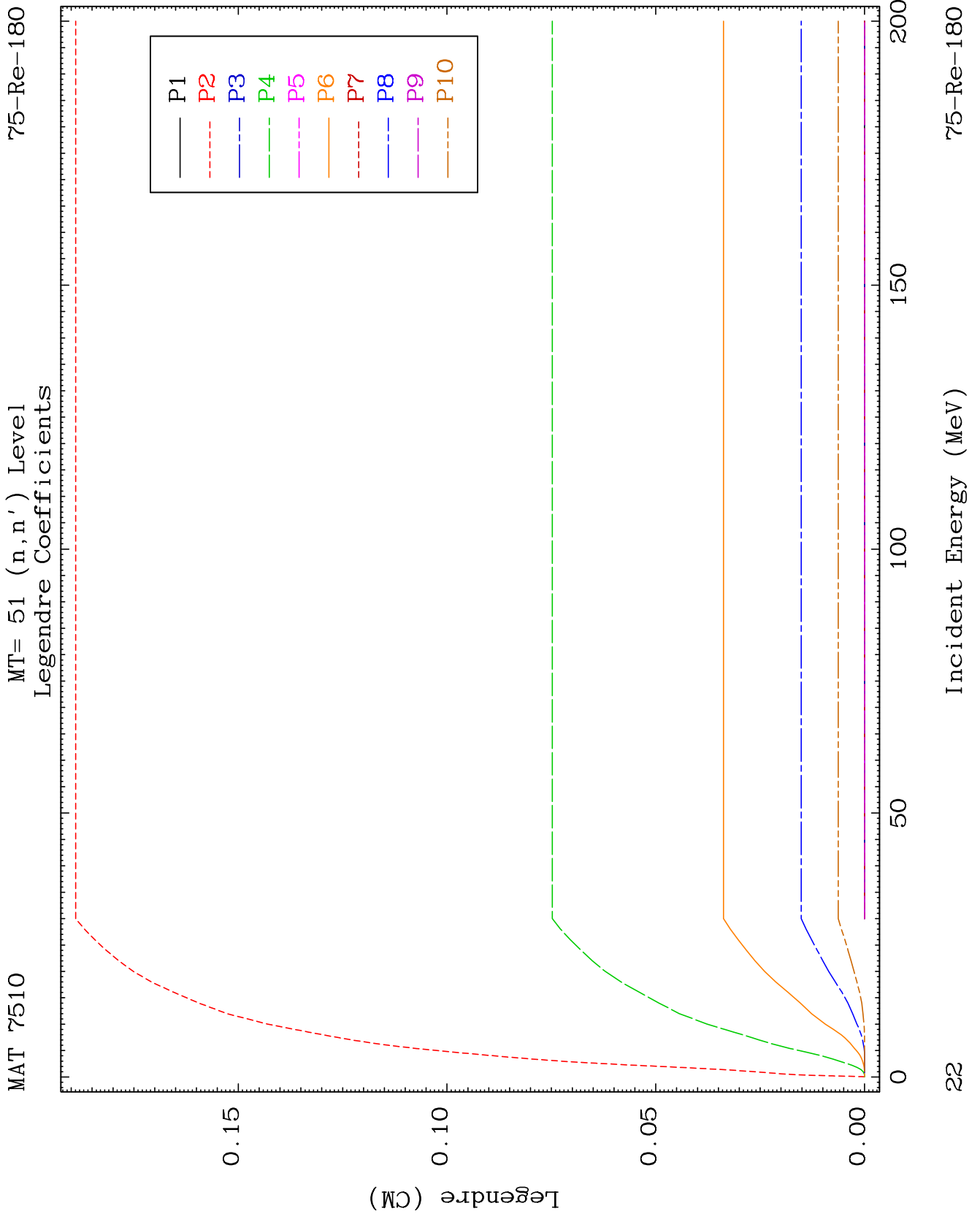


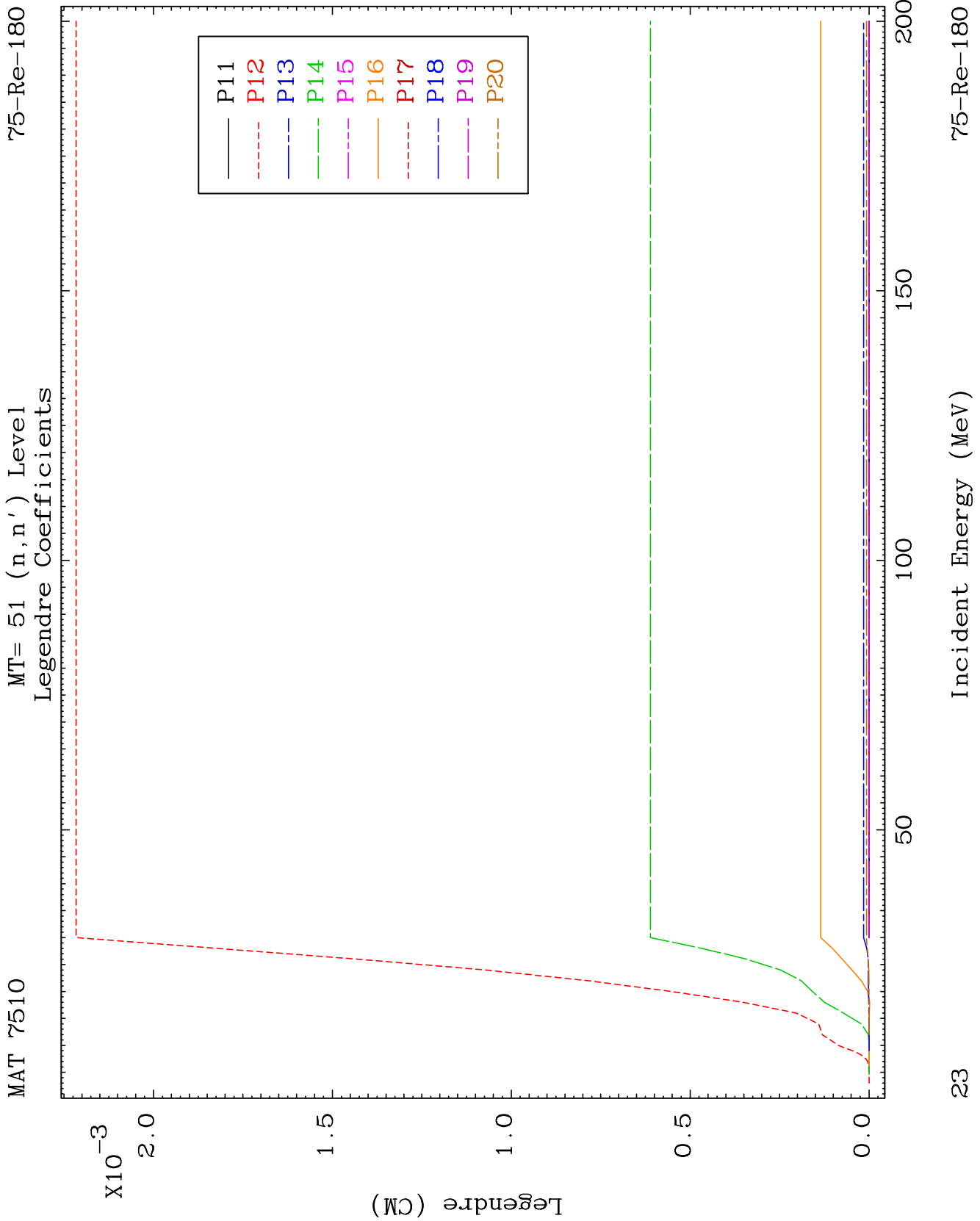




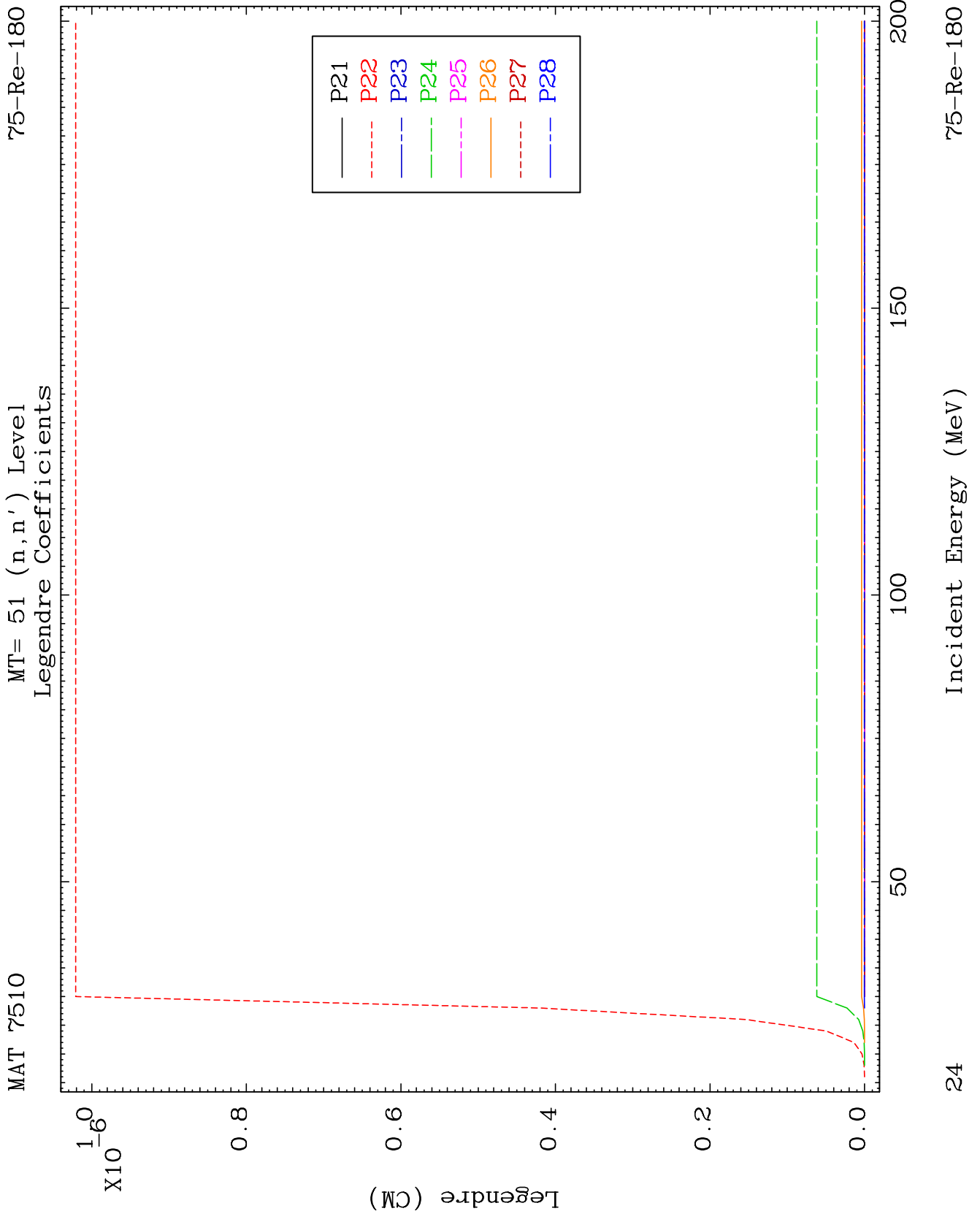


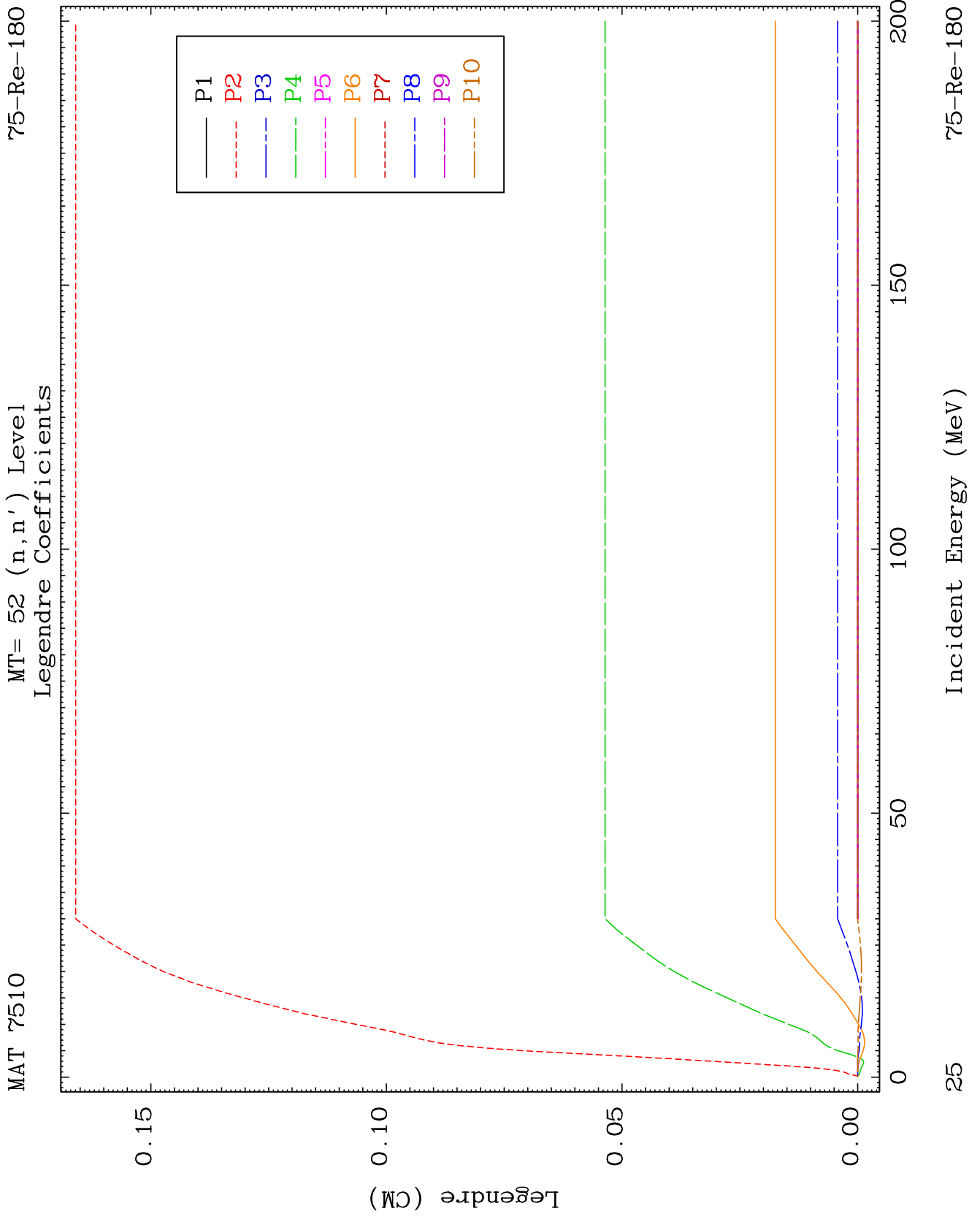








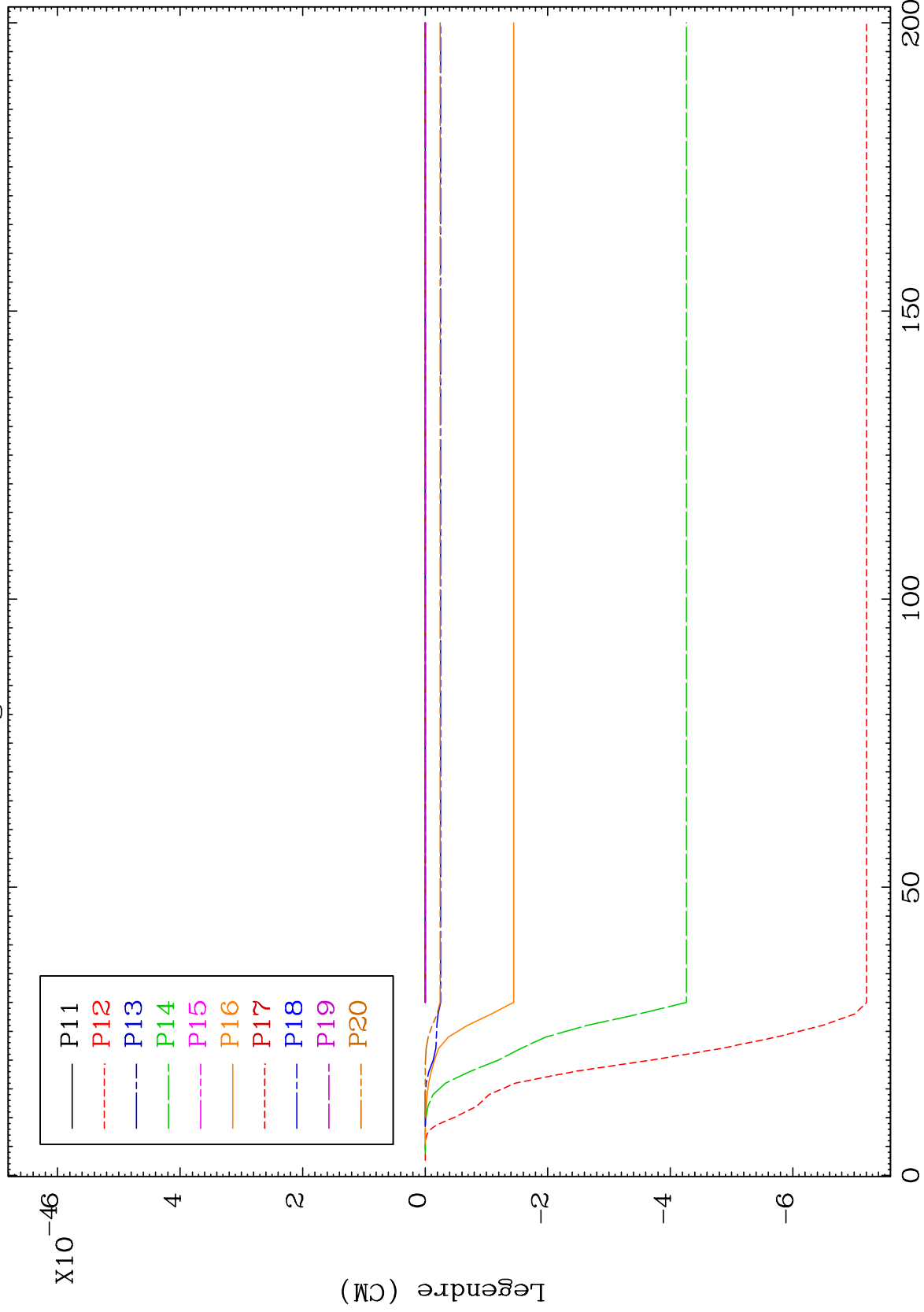




MAT 7510

MT= 52 (n,n') Level  
Legendre Coefficients

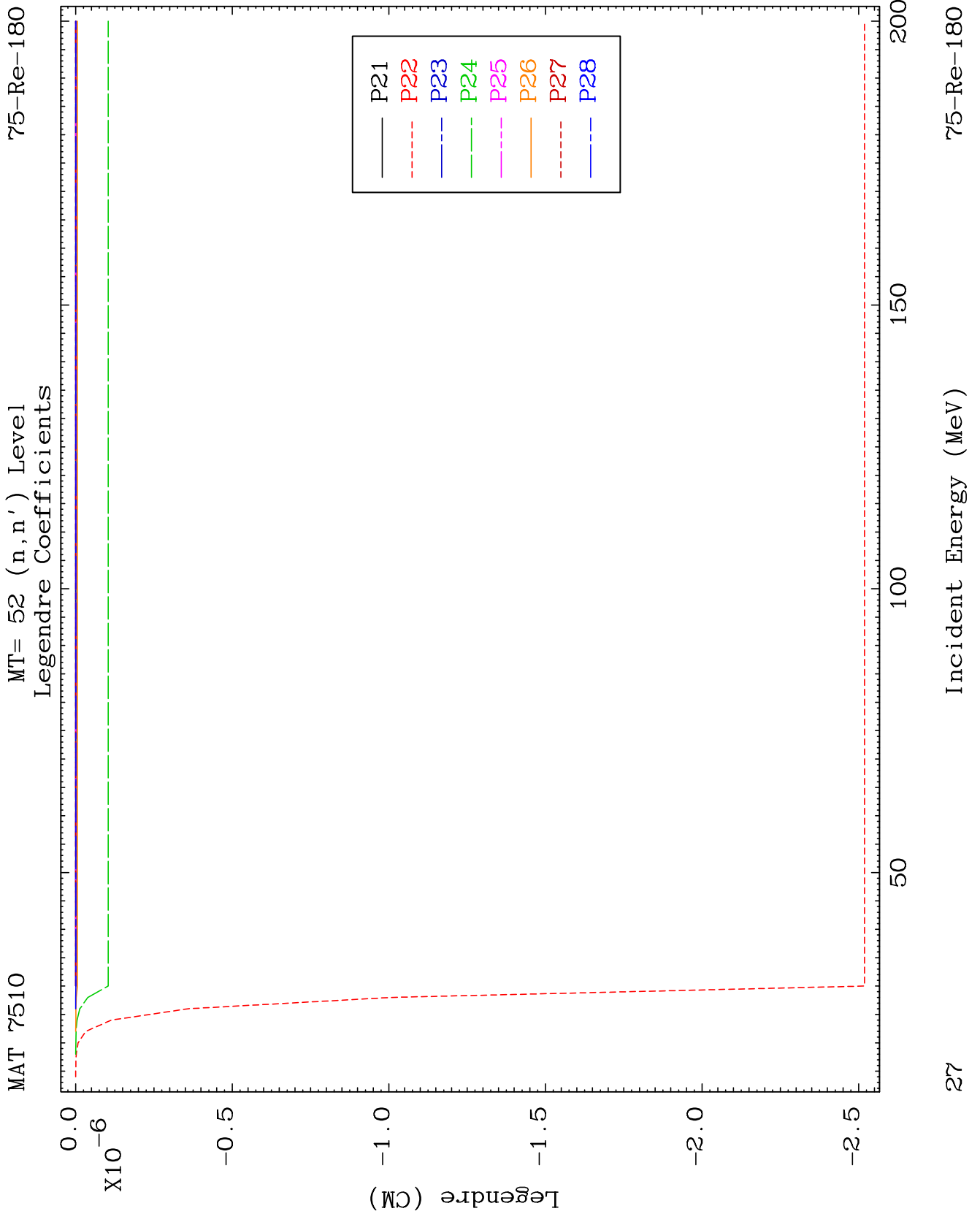
75-Re-180



26

Incident Energy (MeV)

75-Re-180

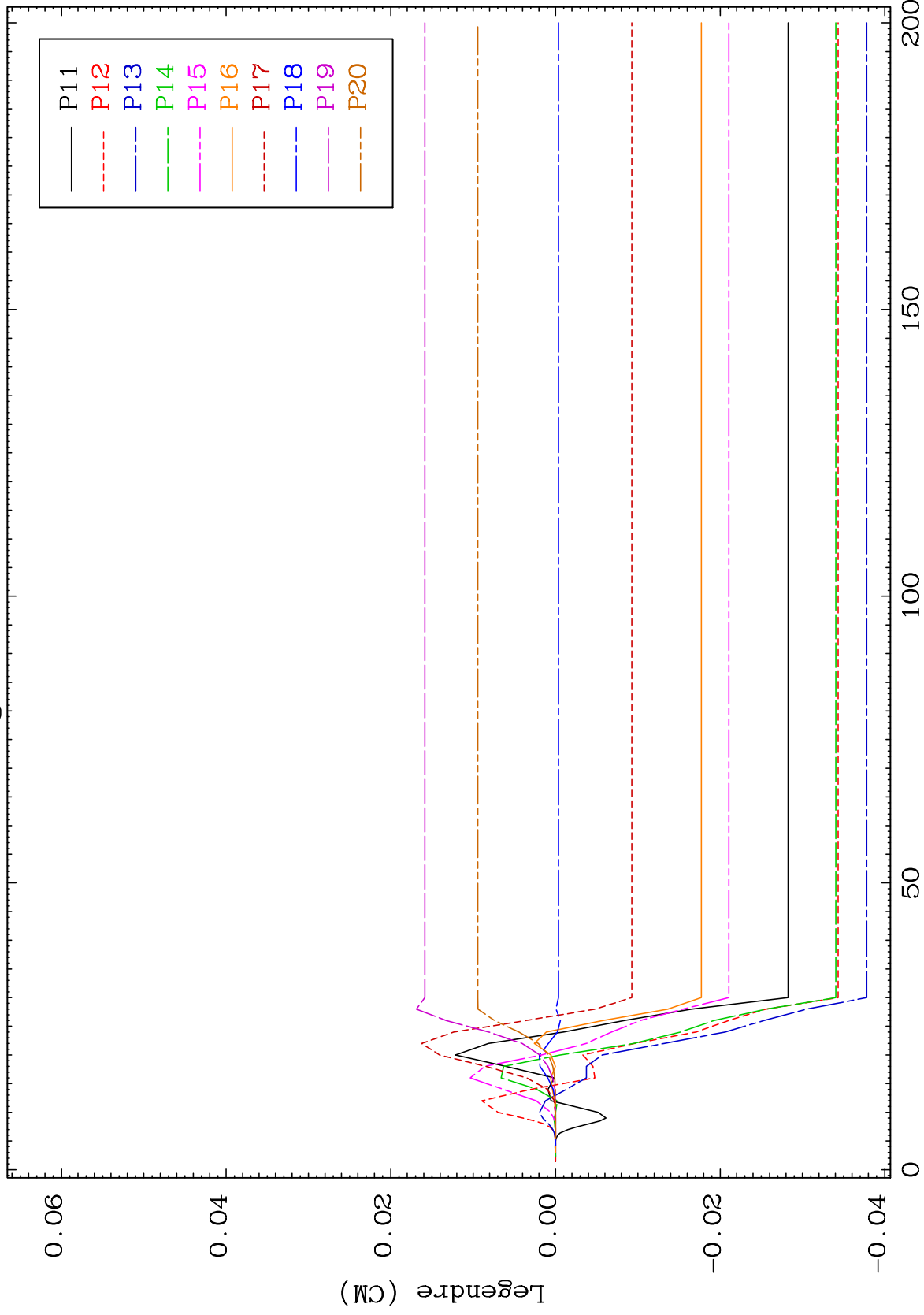




MAT 7510

MT= 53 (n,n') Level  
Legendre Coefficients

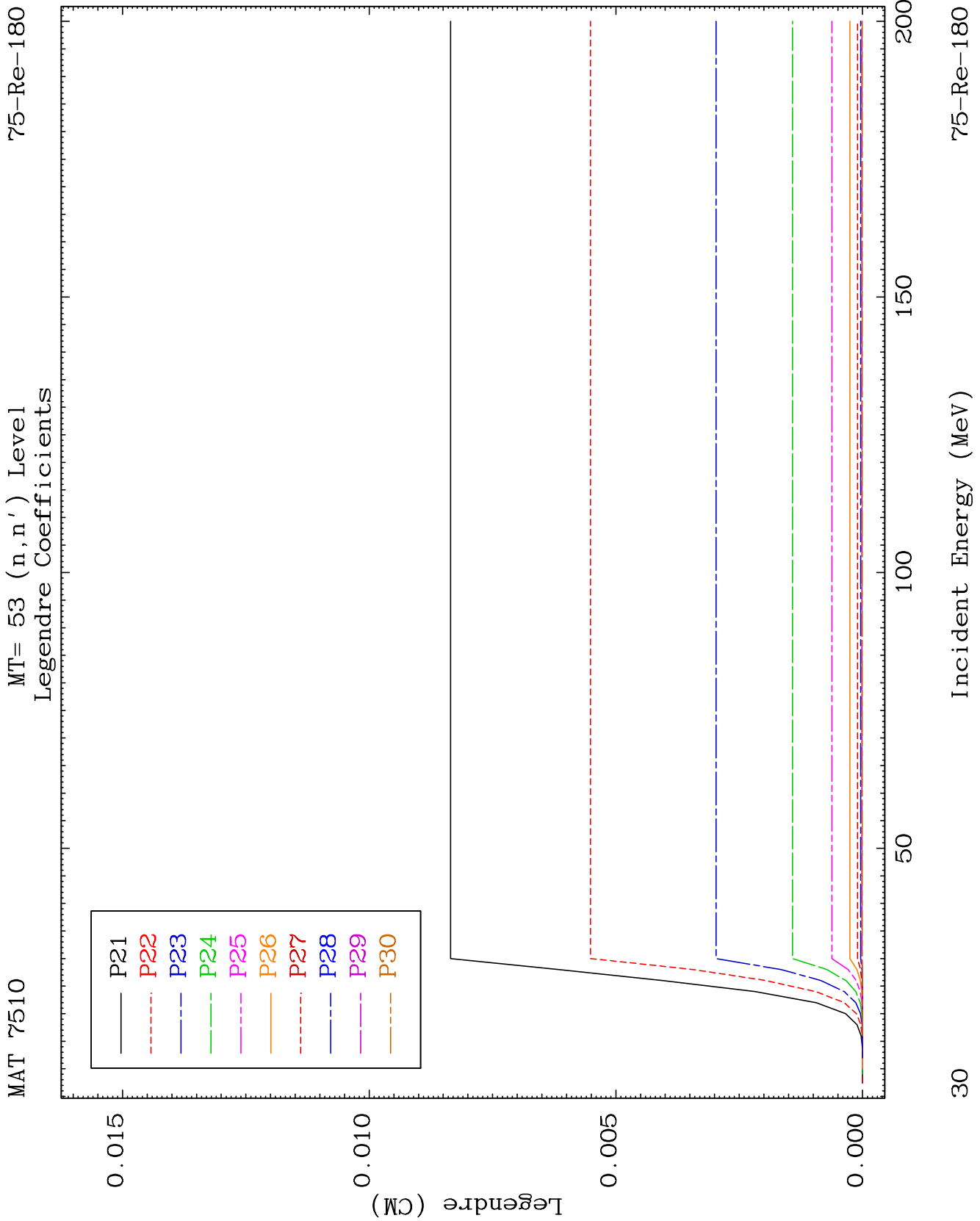
75-Re-180

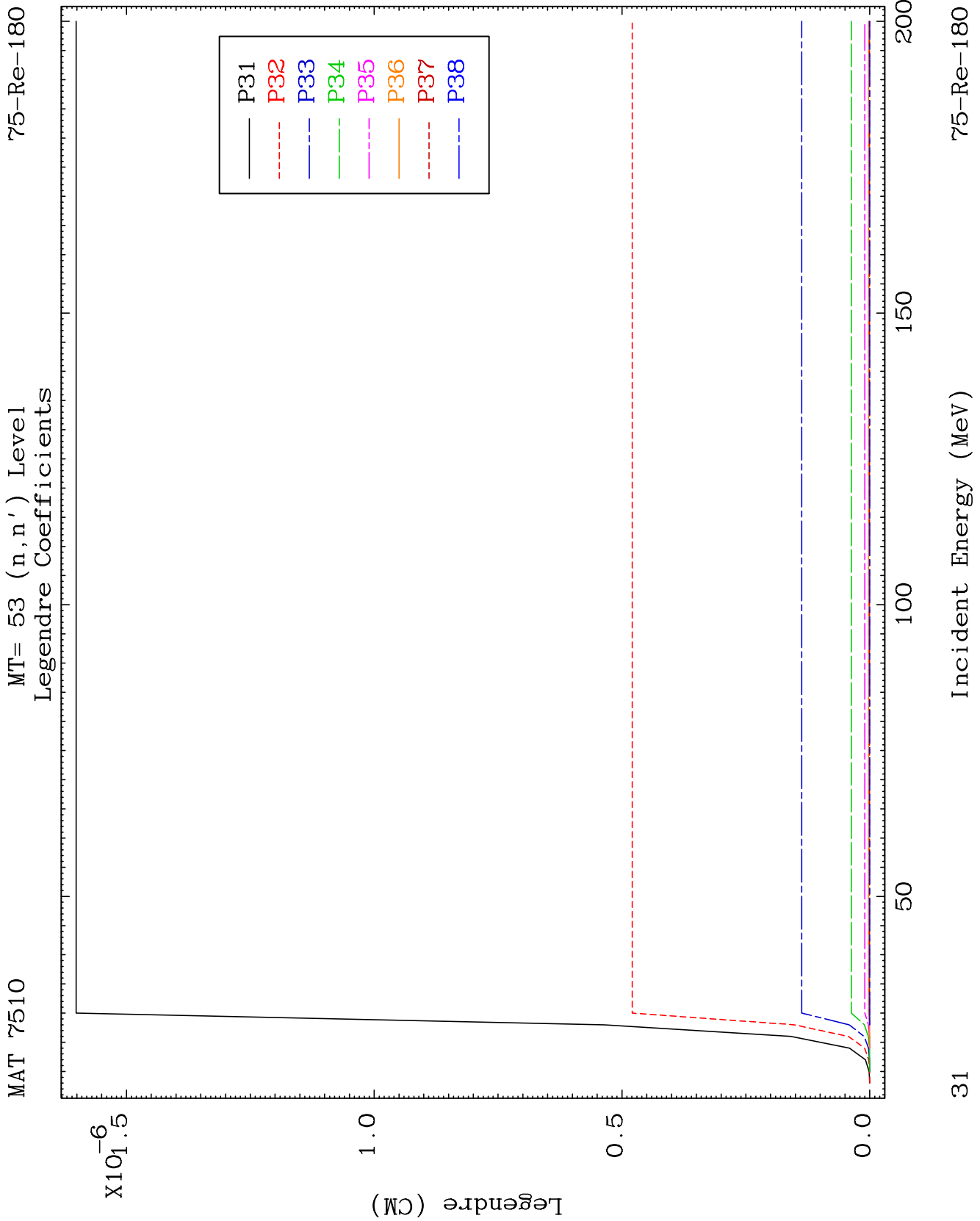


29

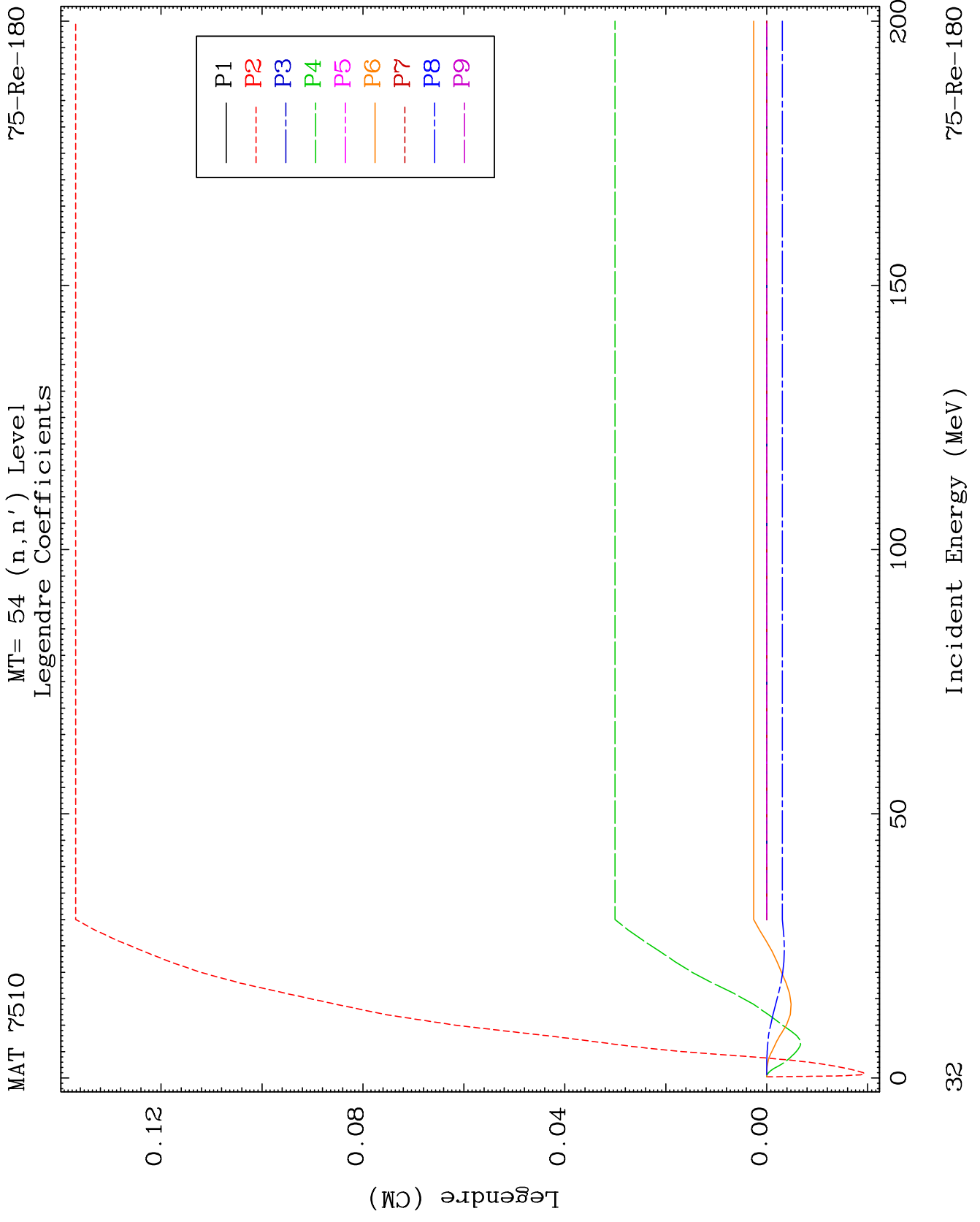
Incident Energy (MeV)

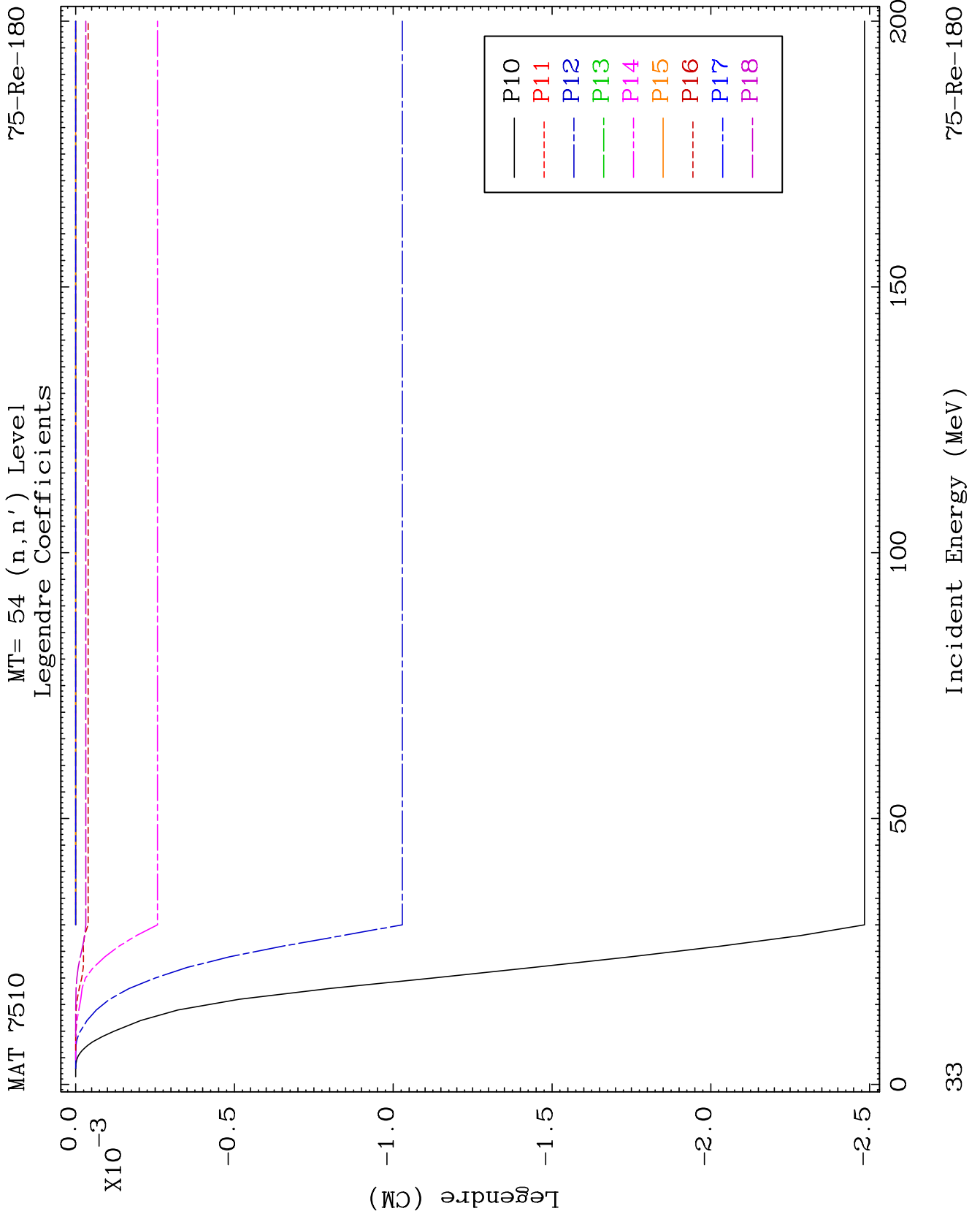
75-Re-180

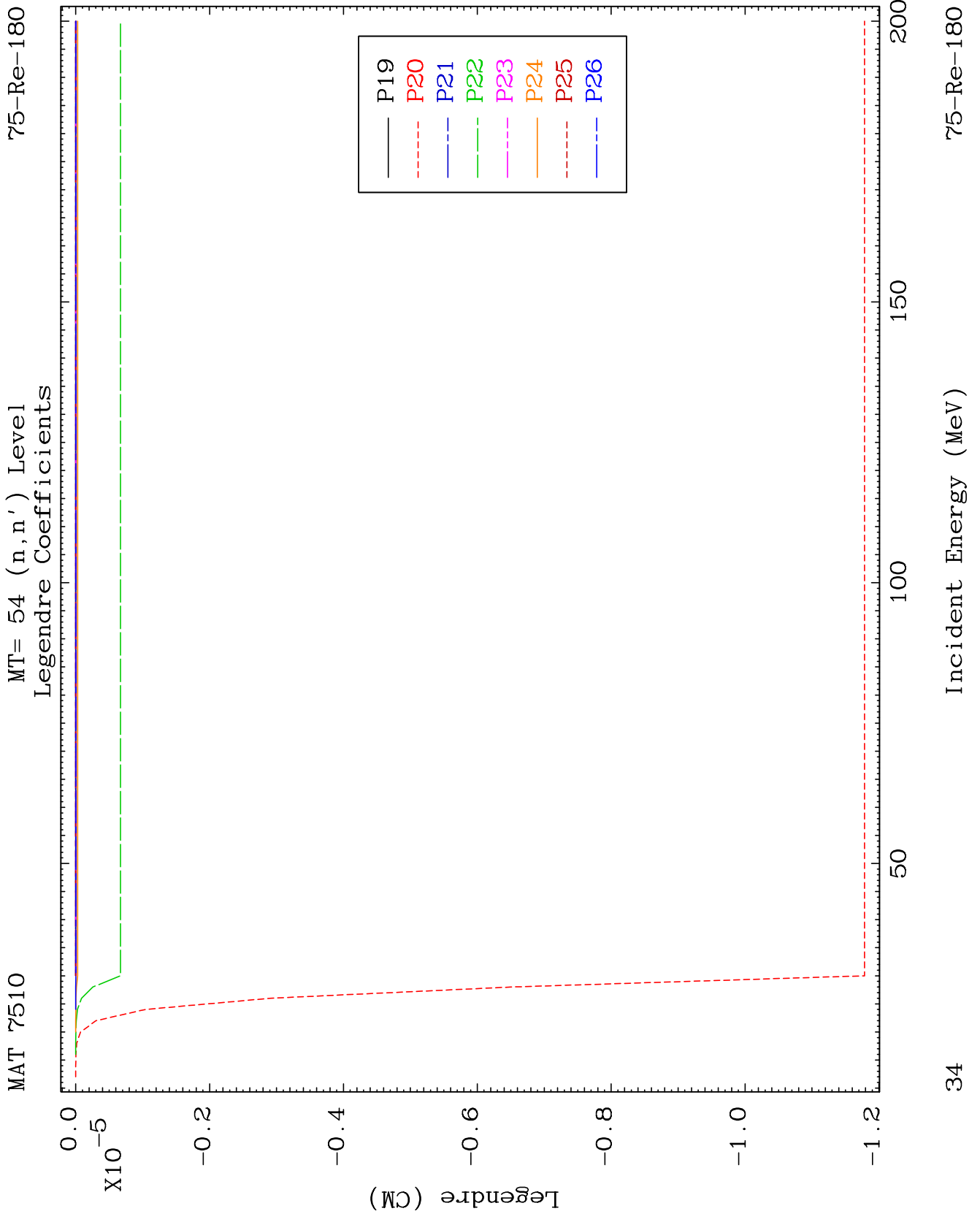










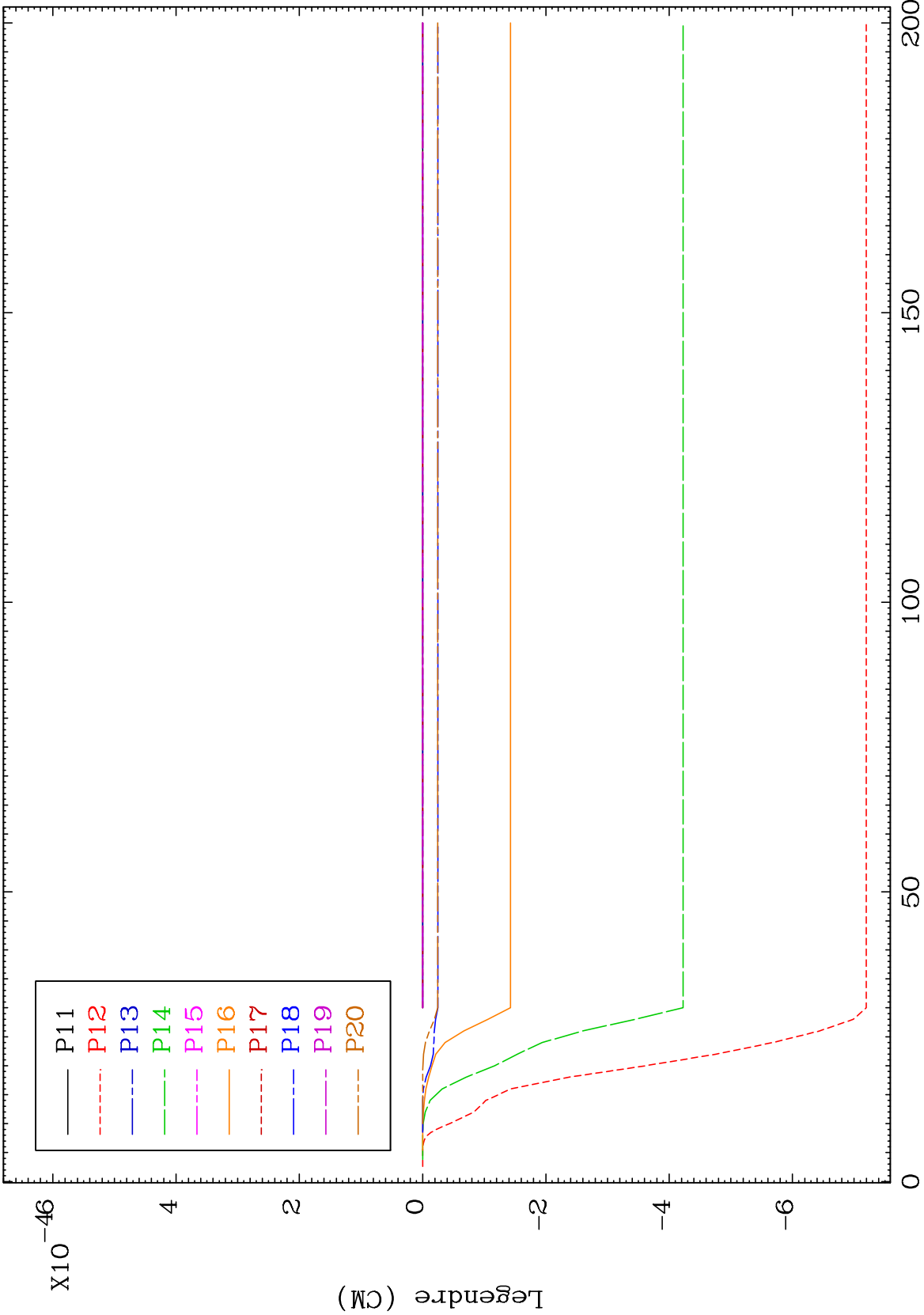
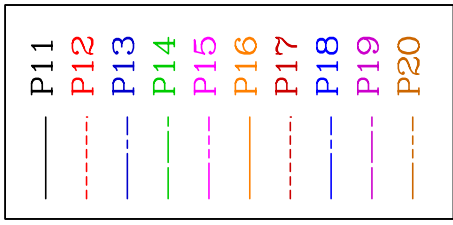




MAT 7510

MT= 55 (n,n') Level  
Legendre Coefficients

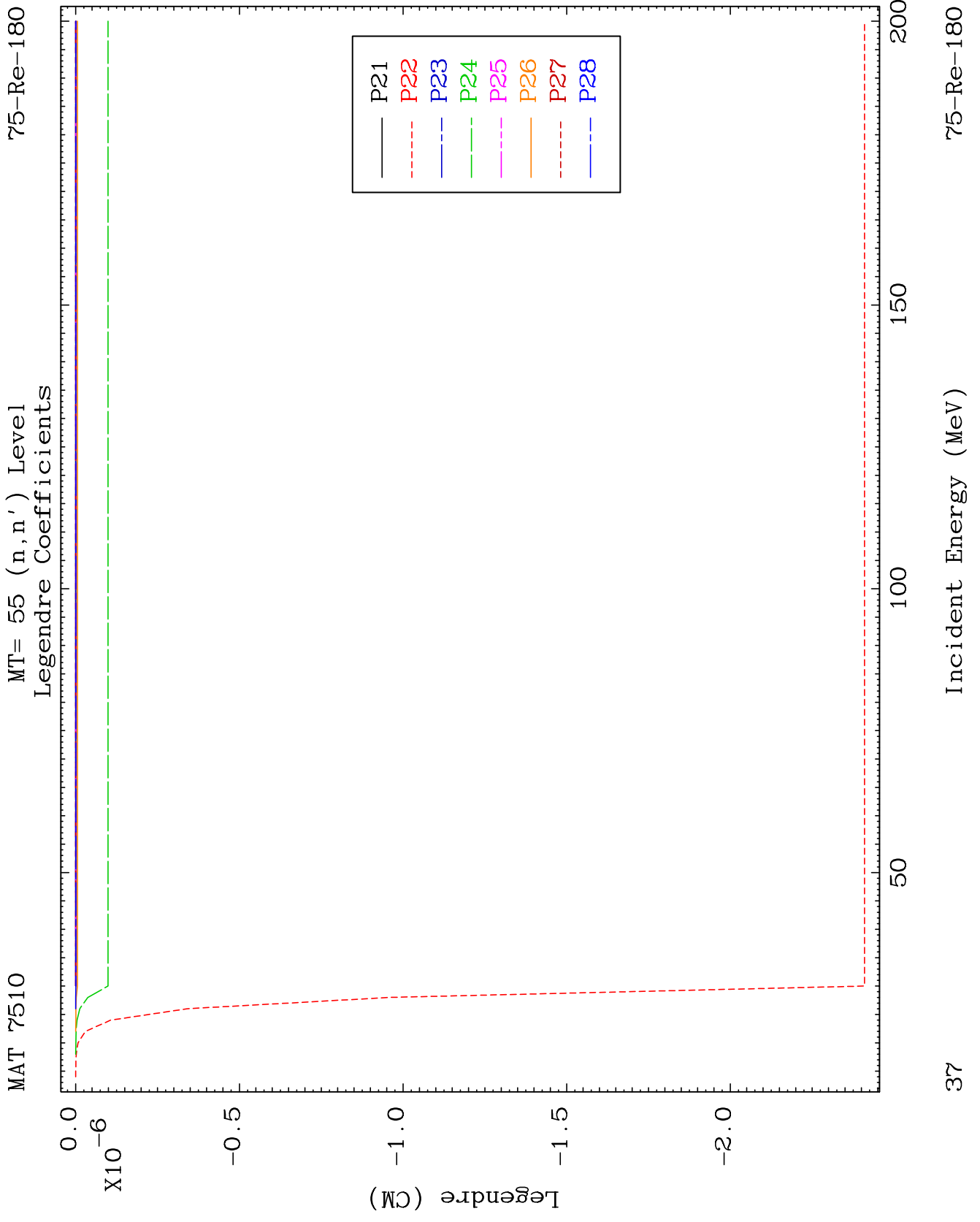
75-Re-180



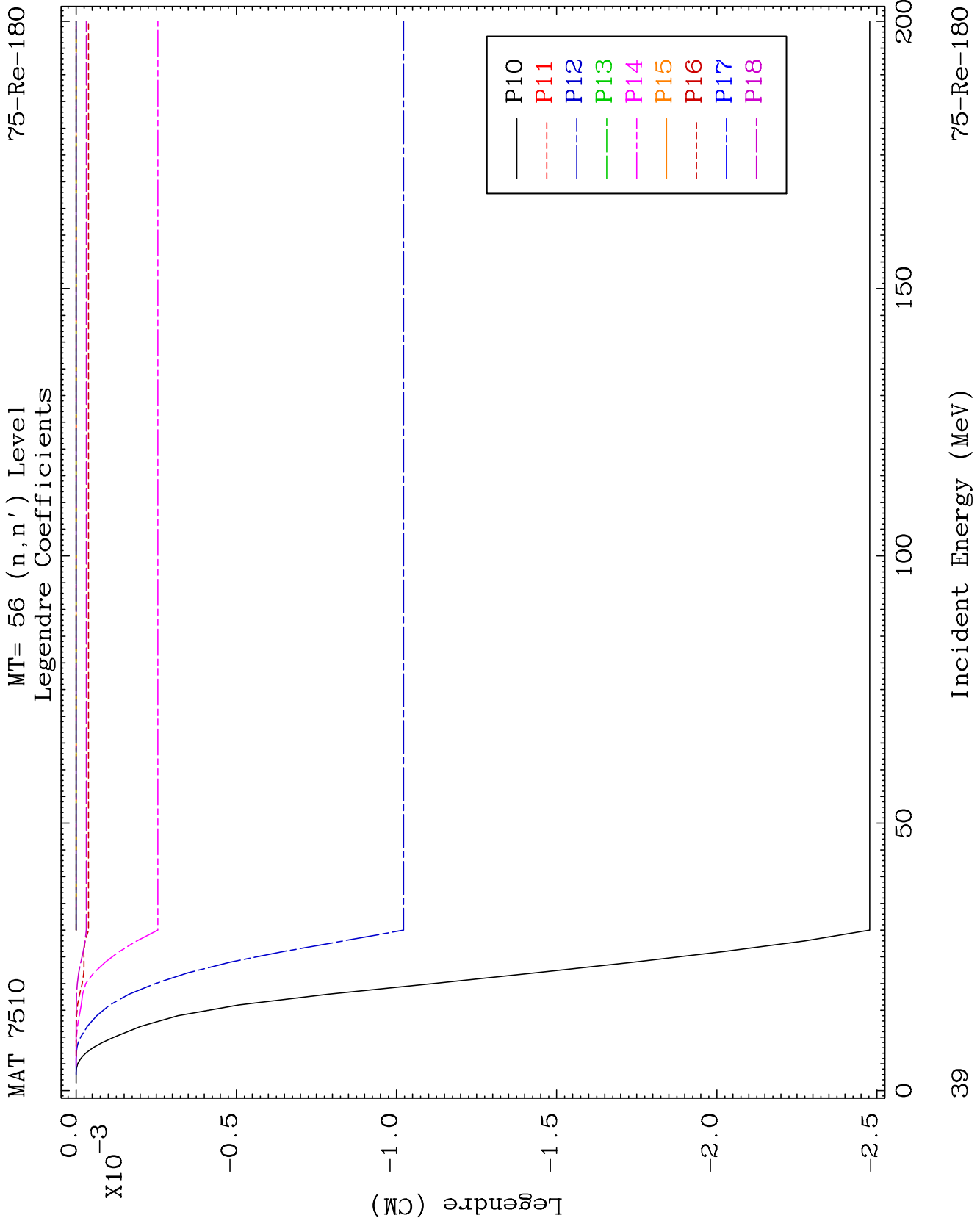
36

Incident Energy (MeV)

75-Re-180

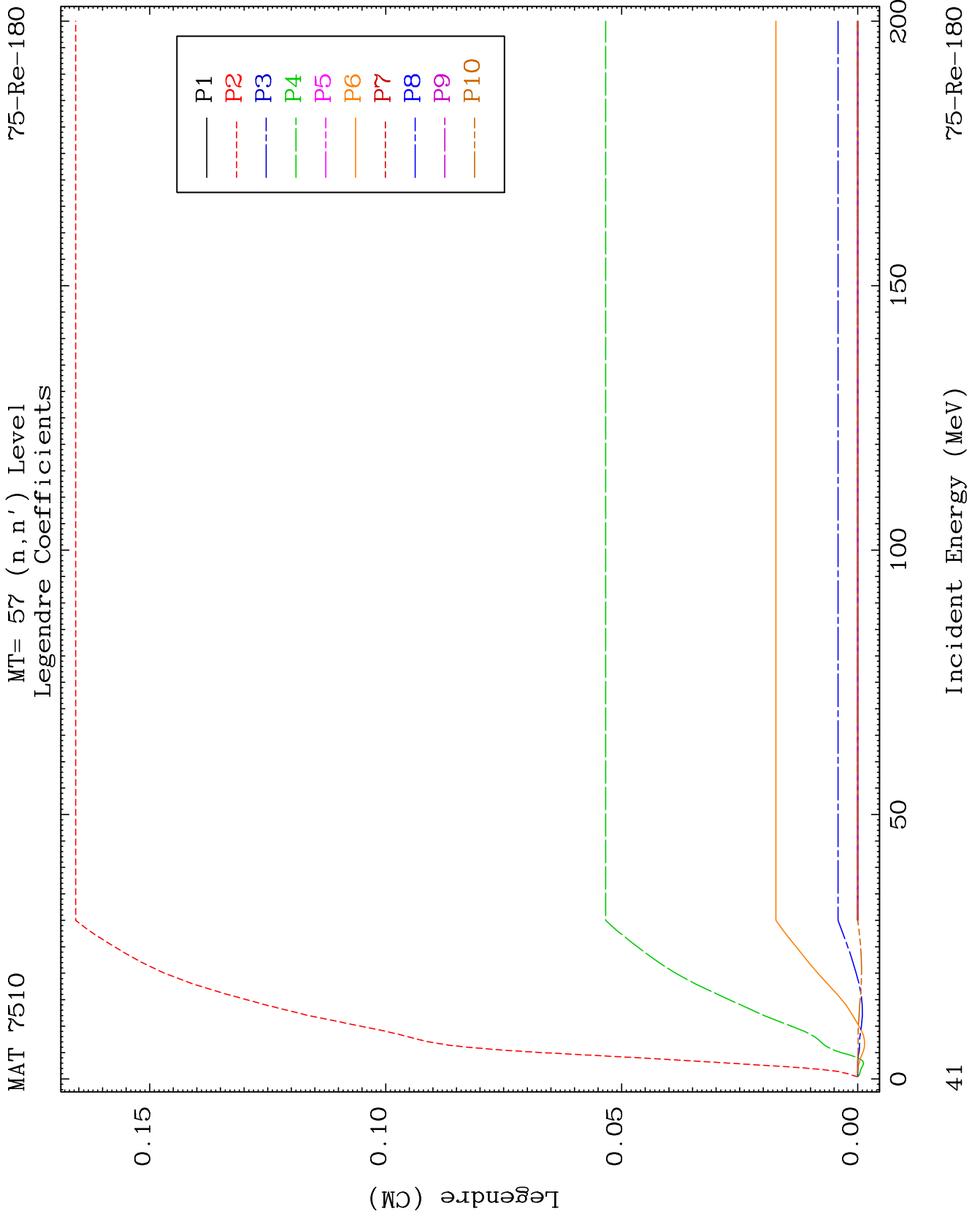


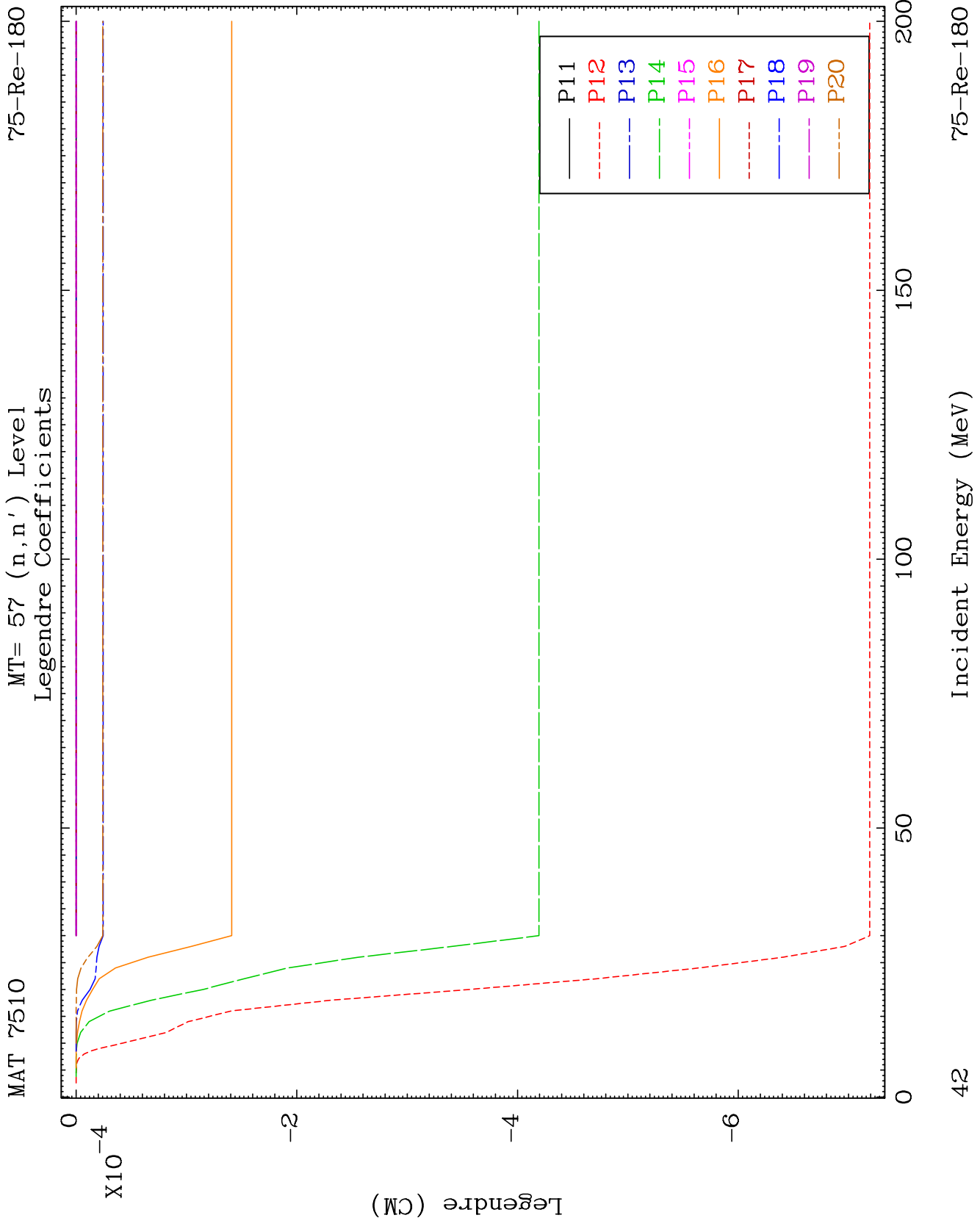


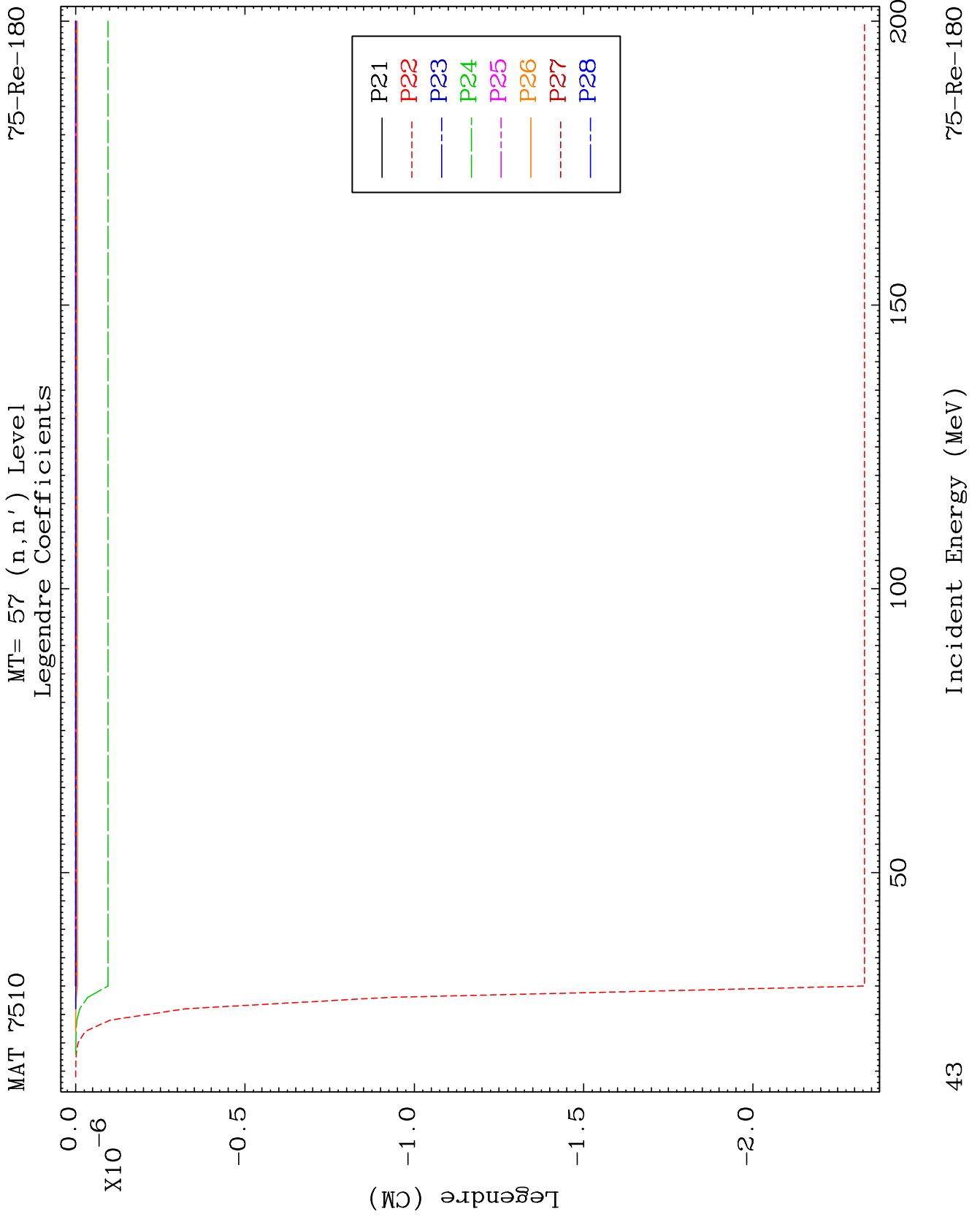


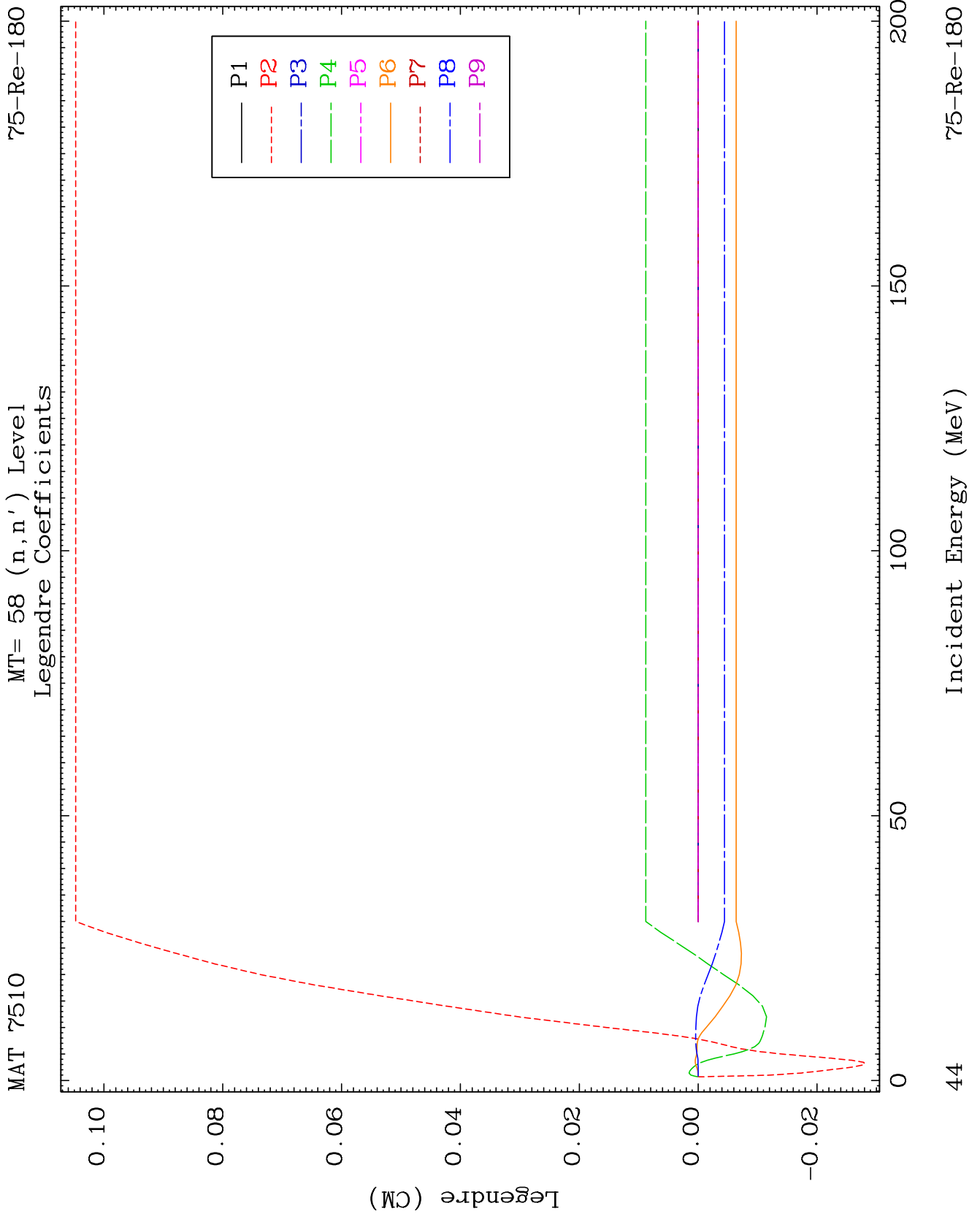


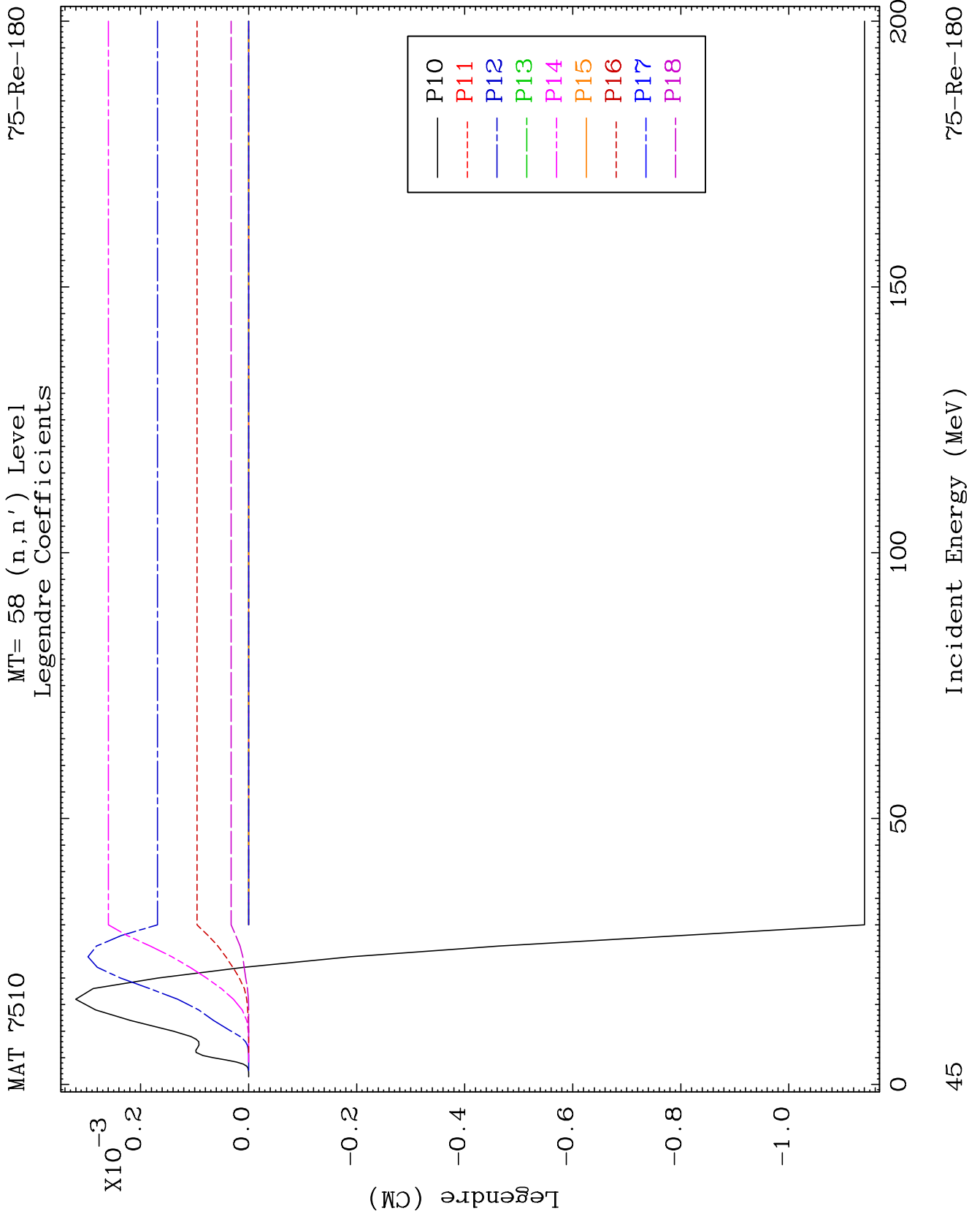


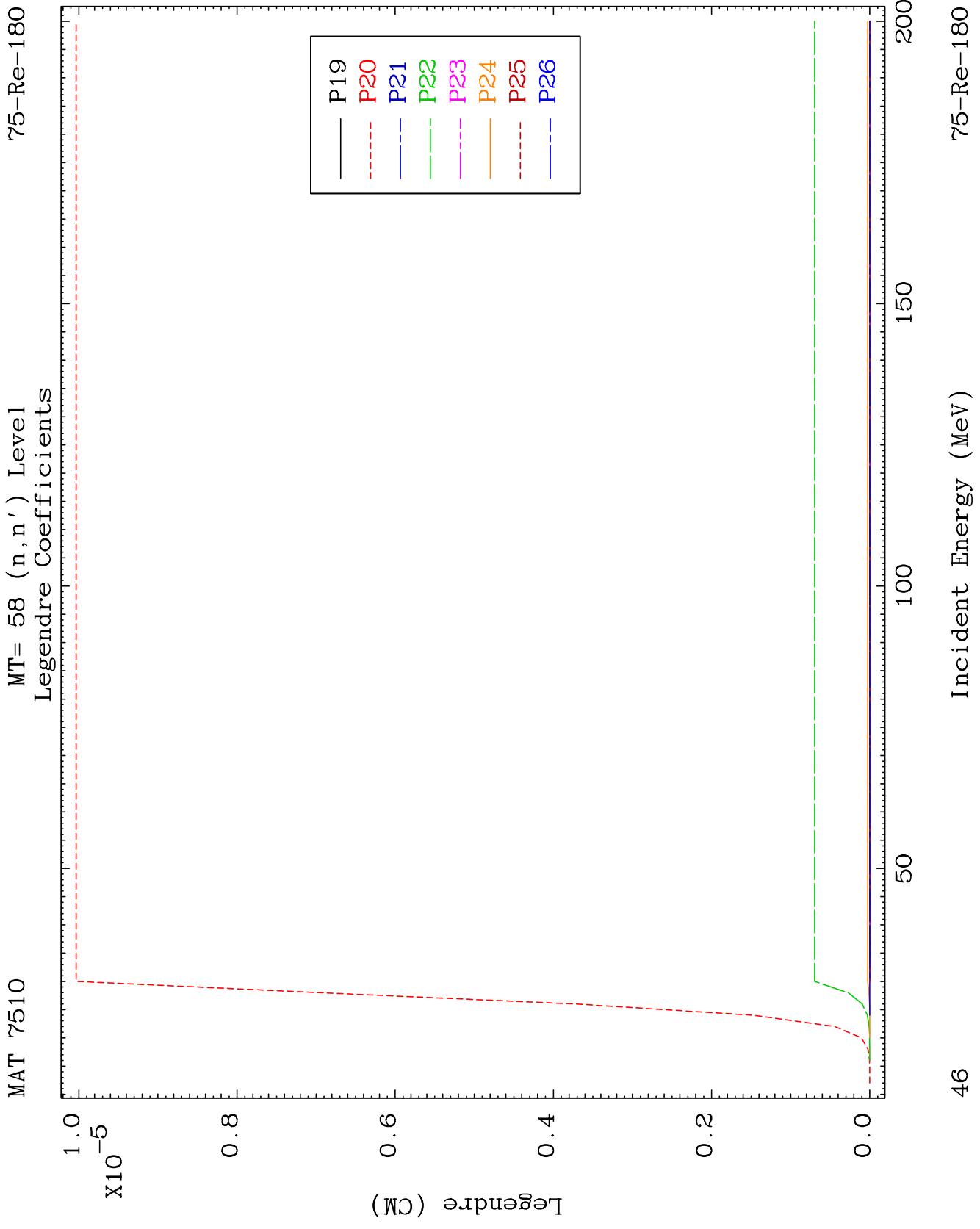


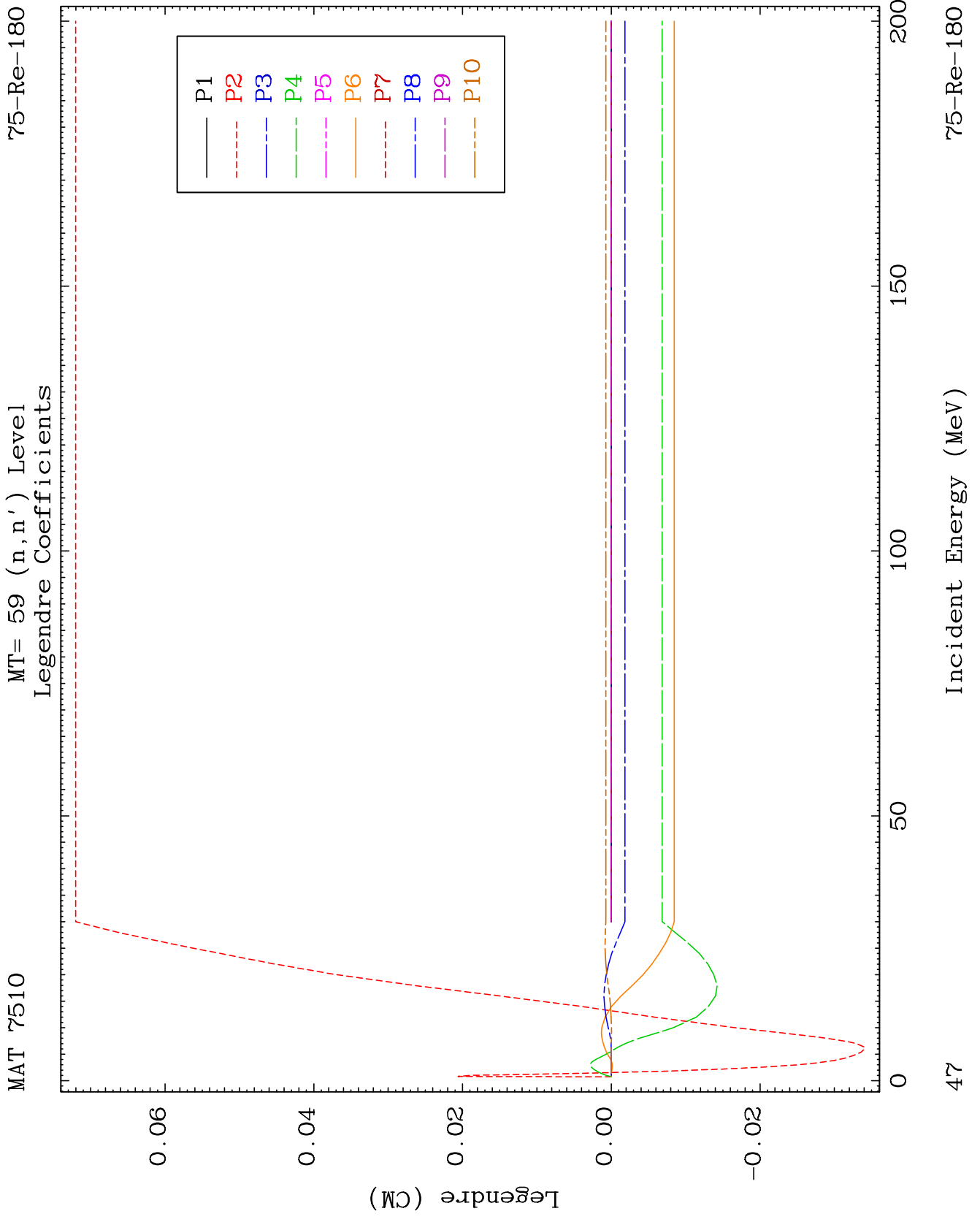




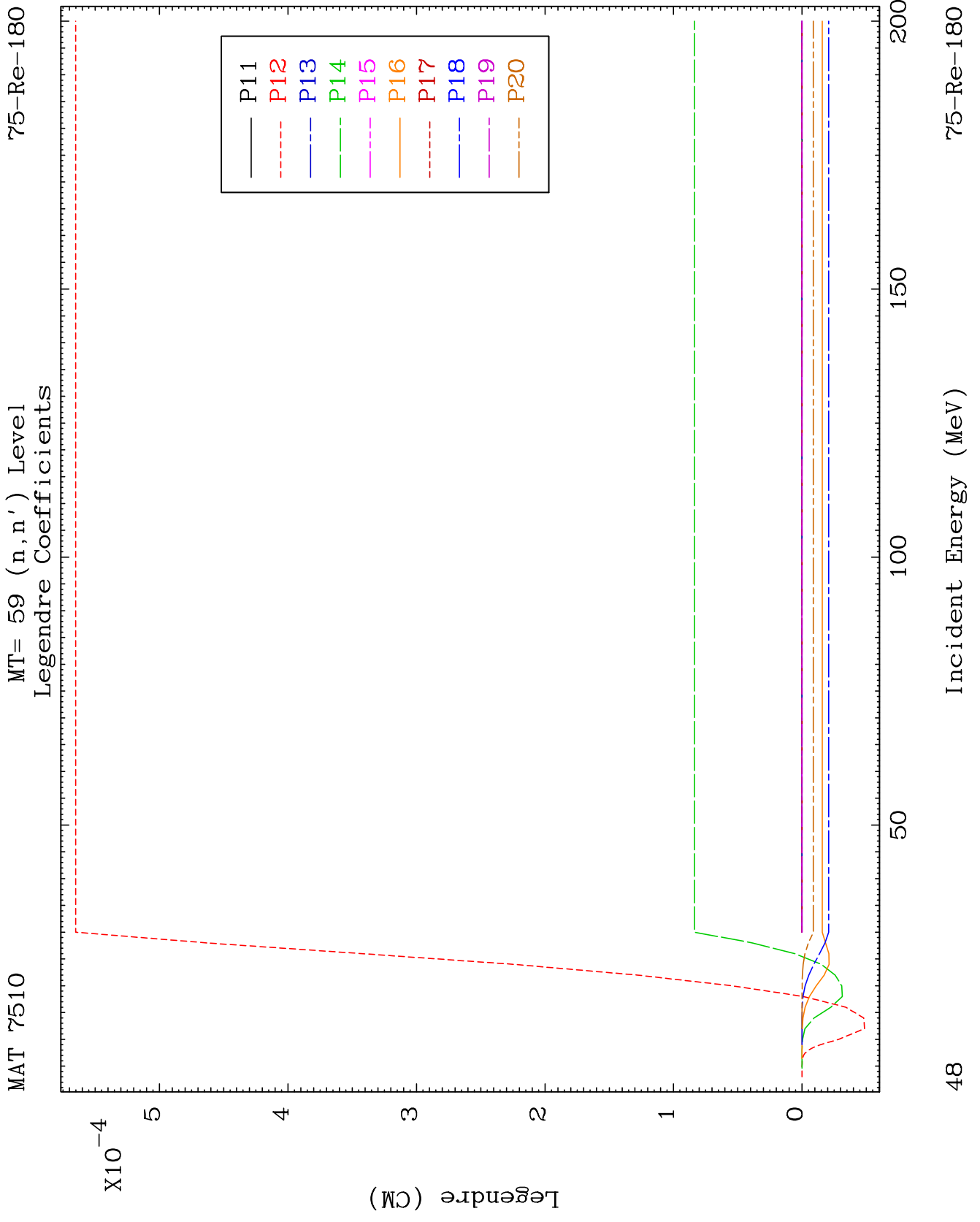


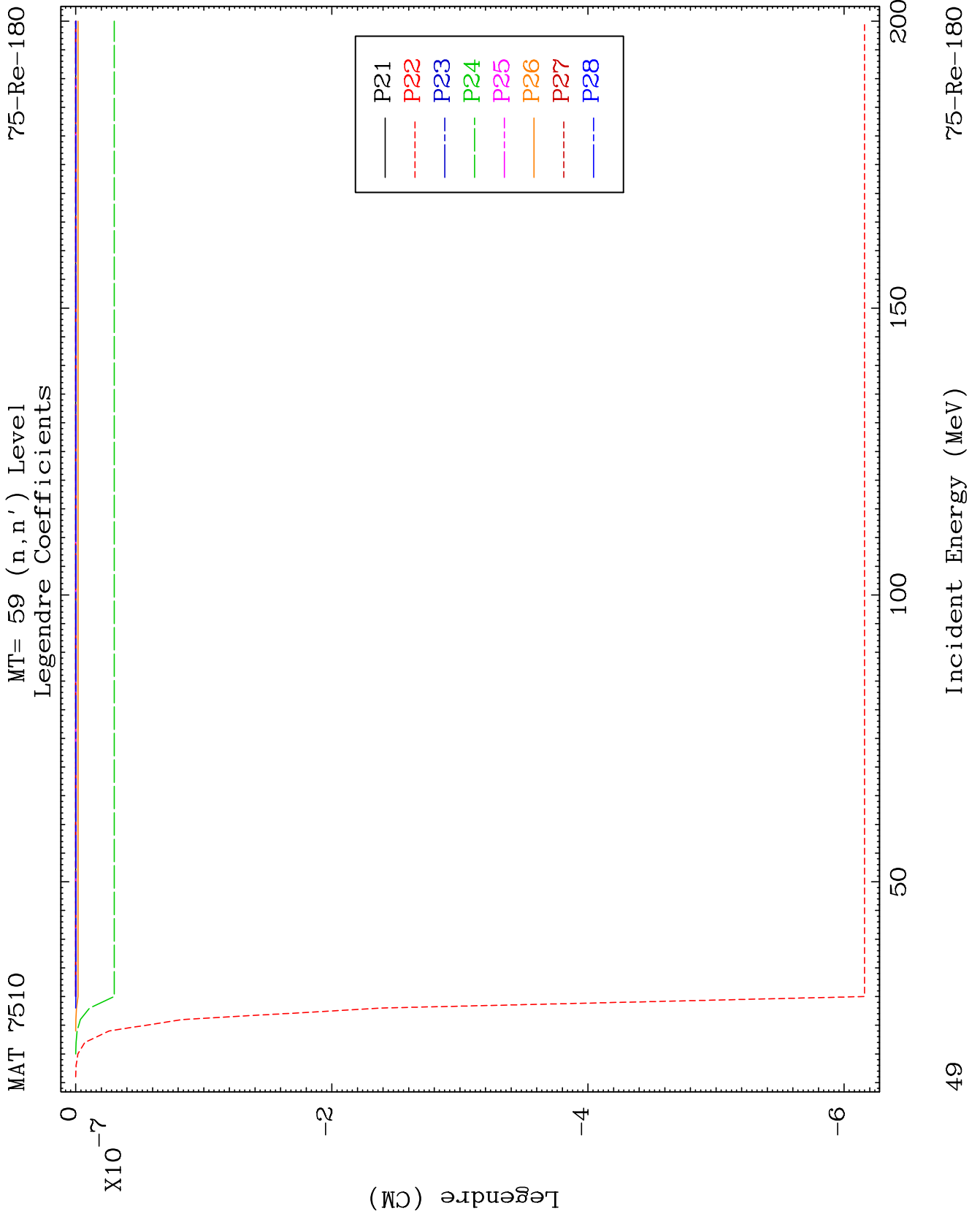


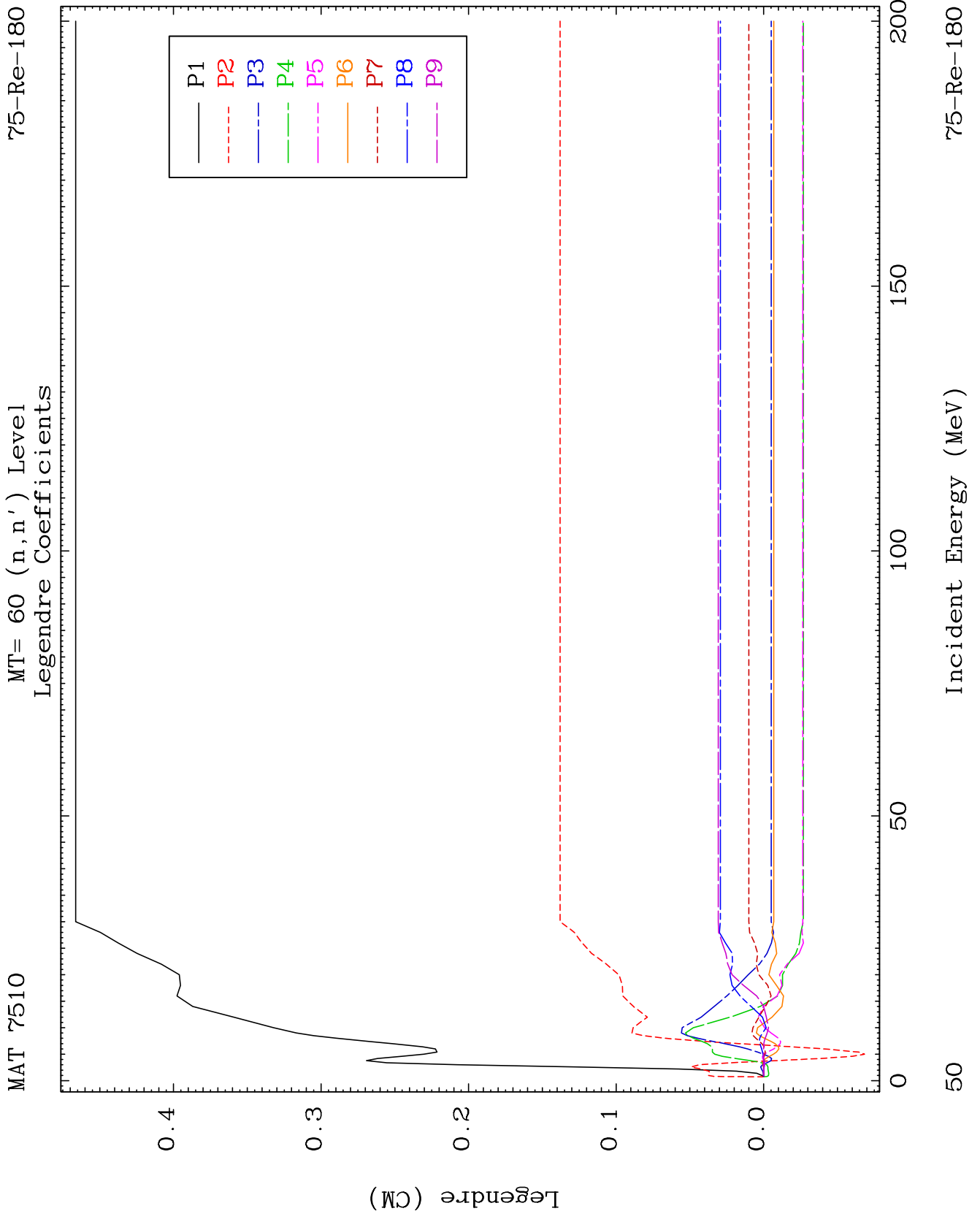


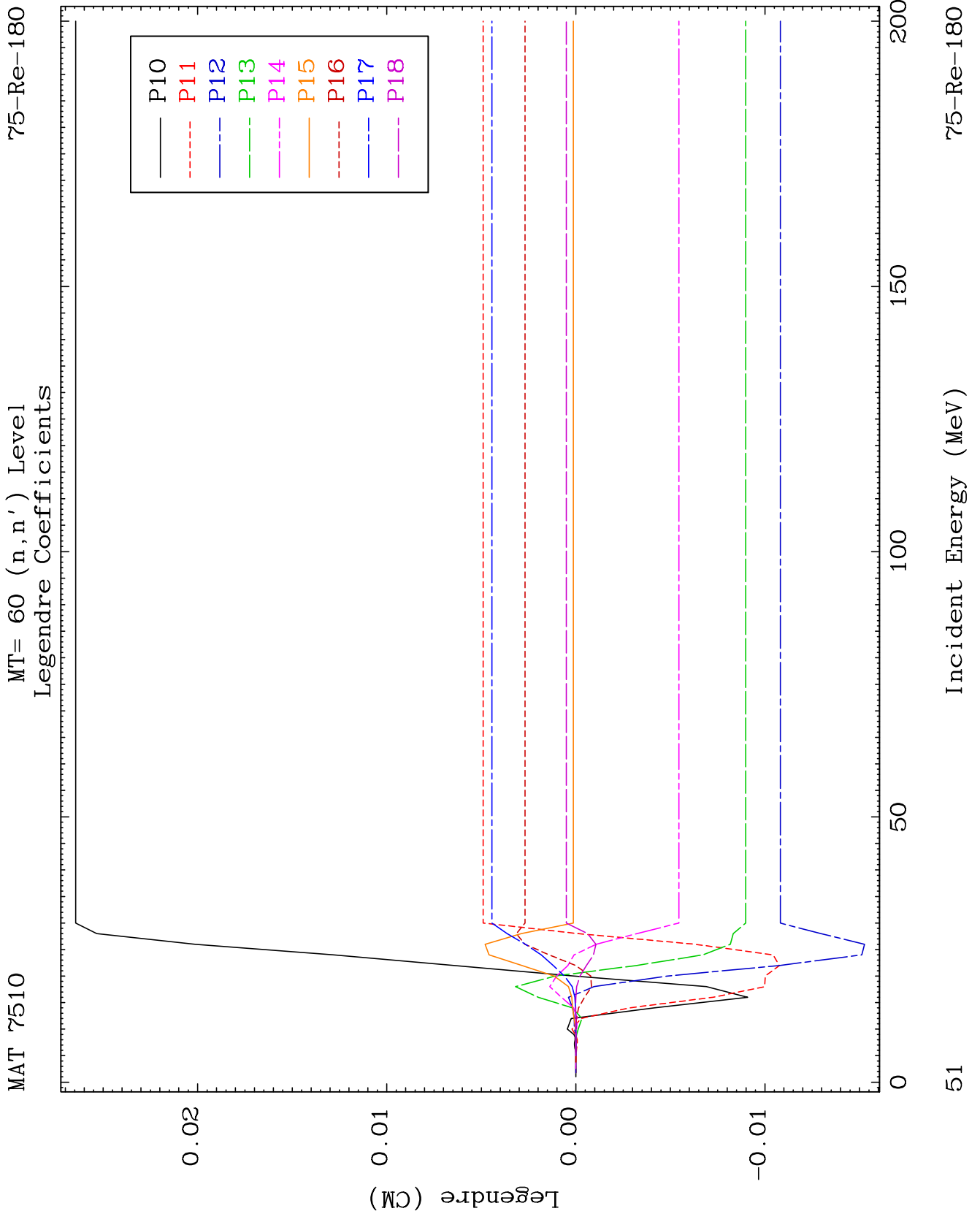








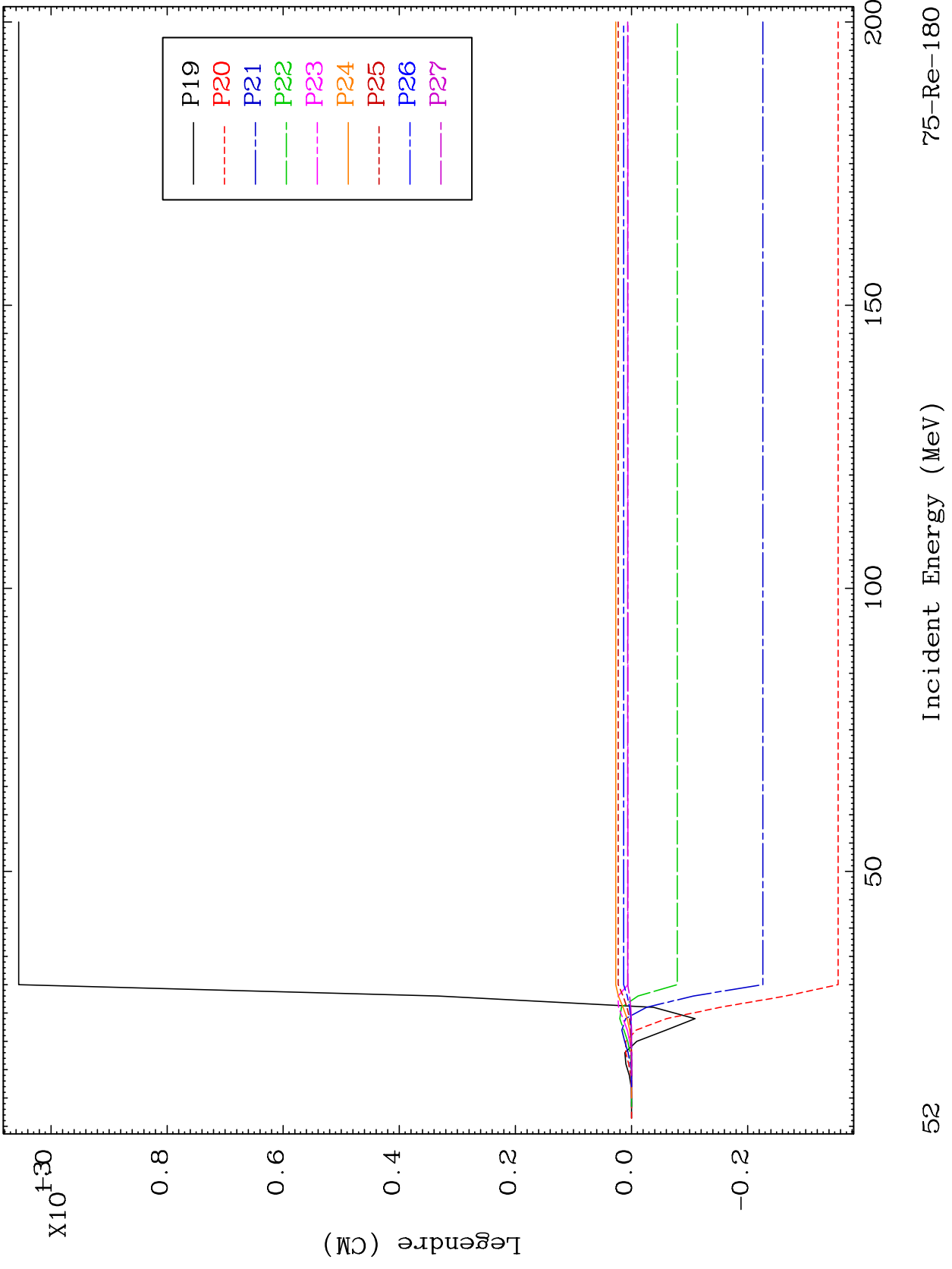


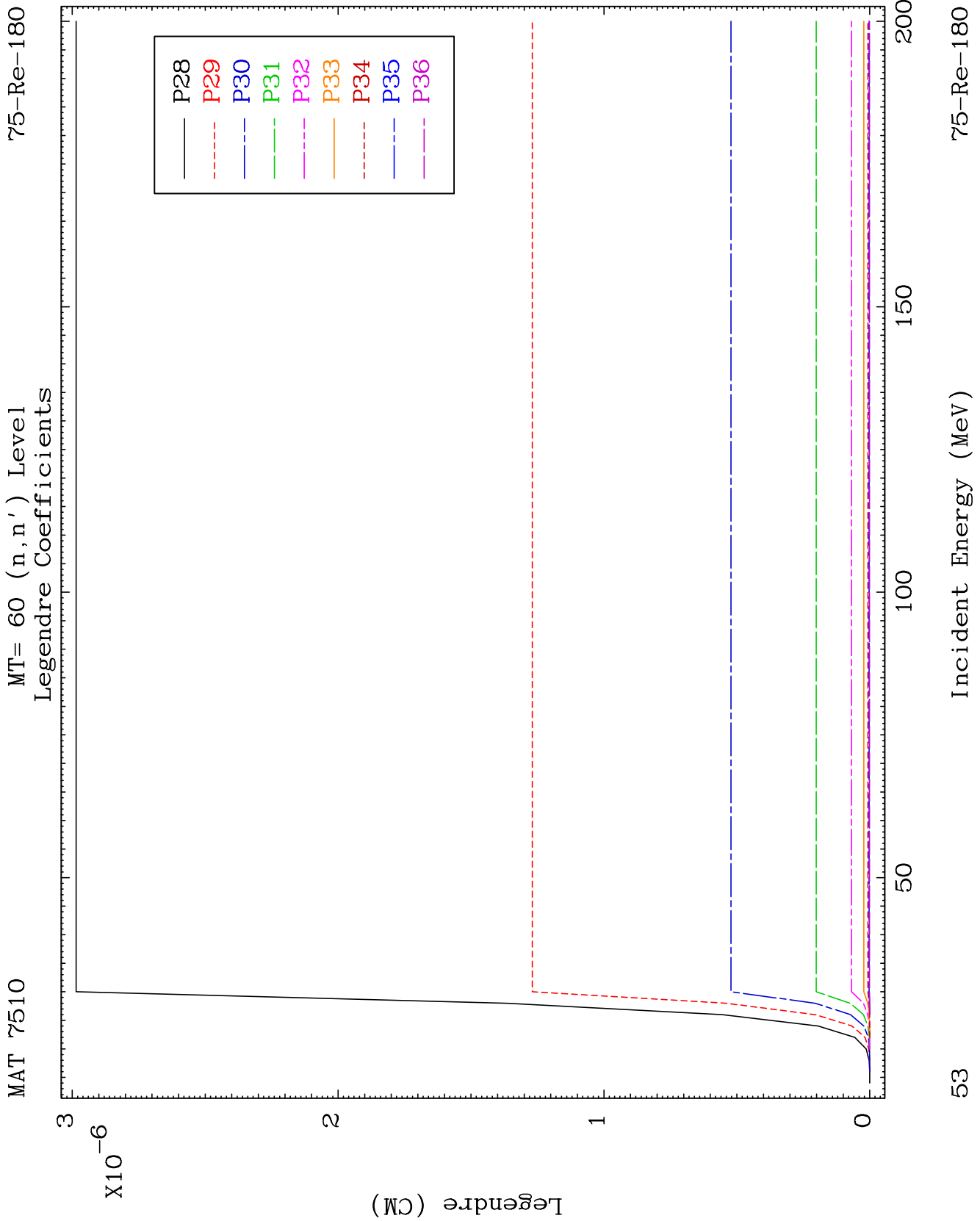


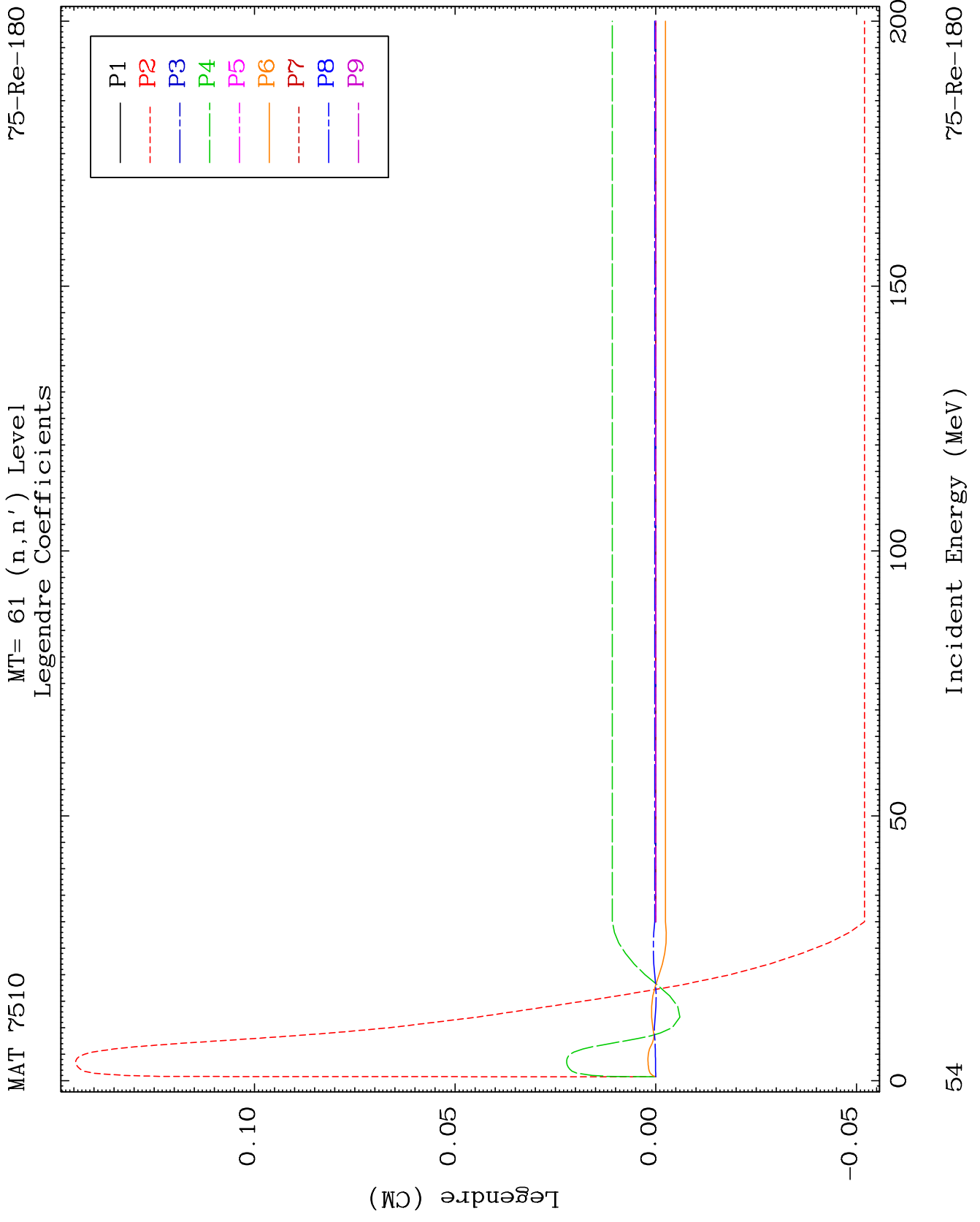
MAT 7510

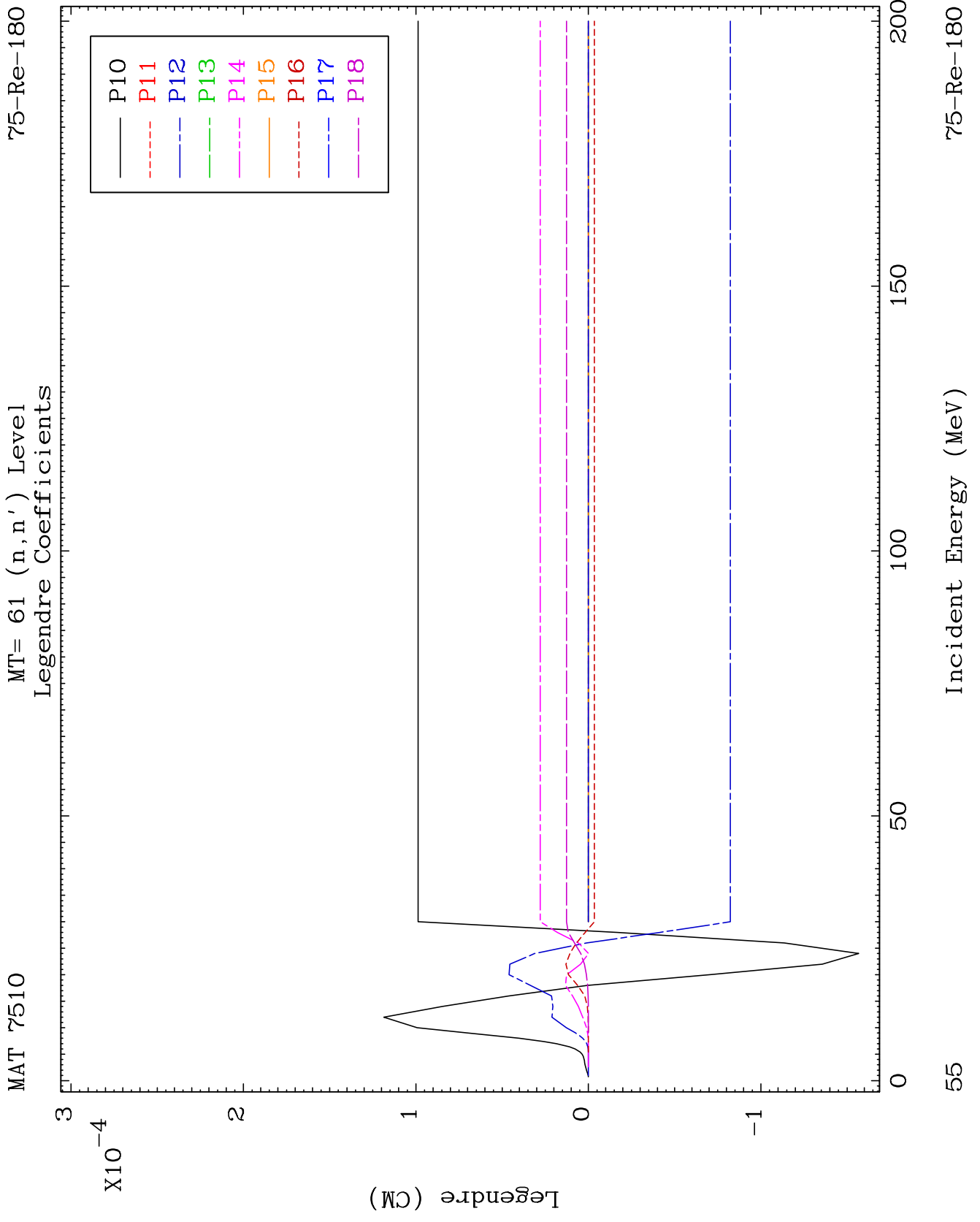
MT= 60 (n,n') Level  
Legendre Coefficients

75-Re-180

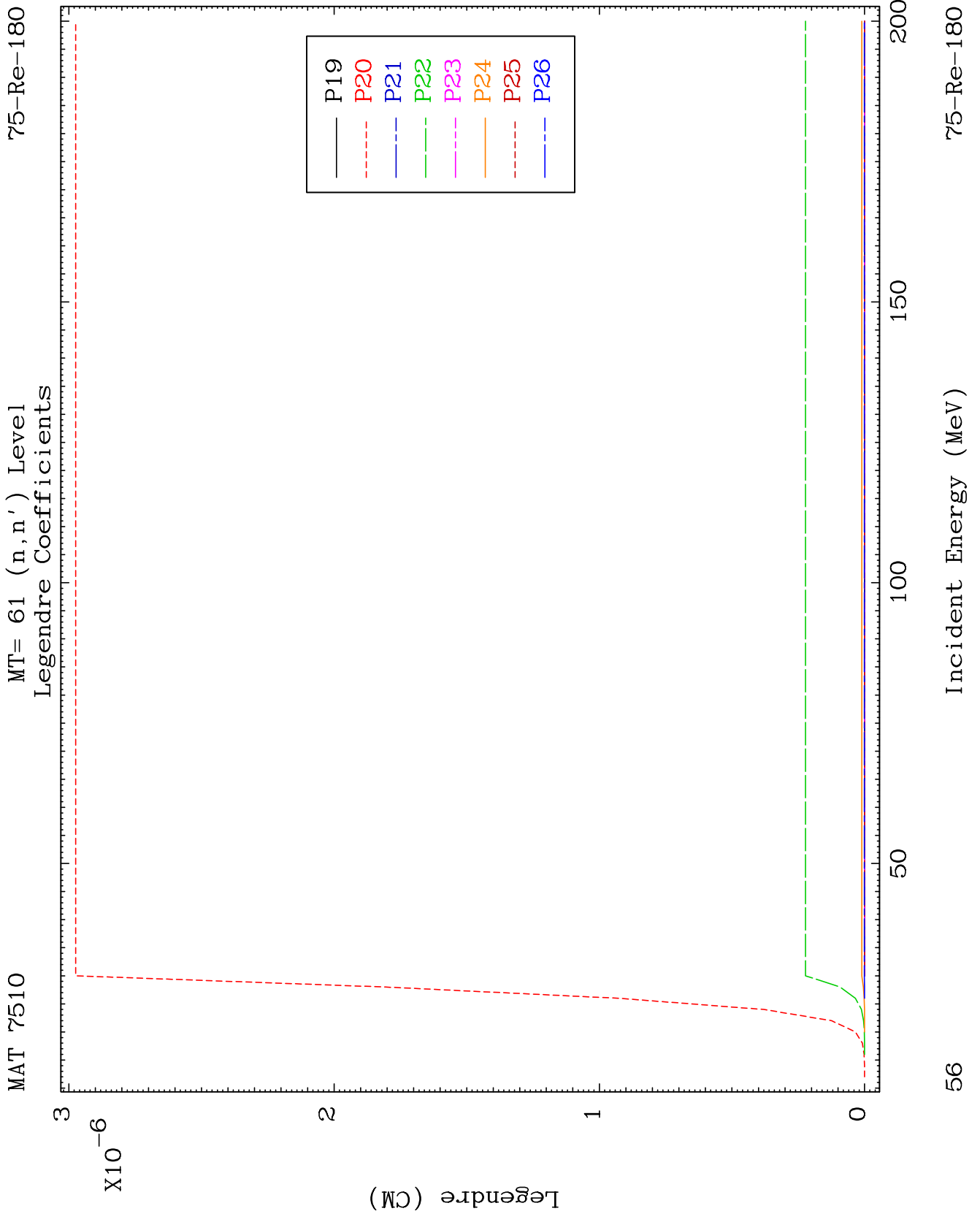


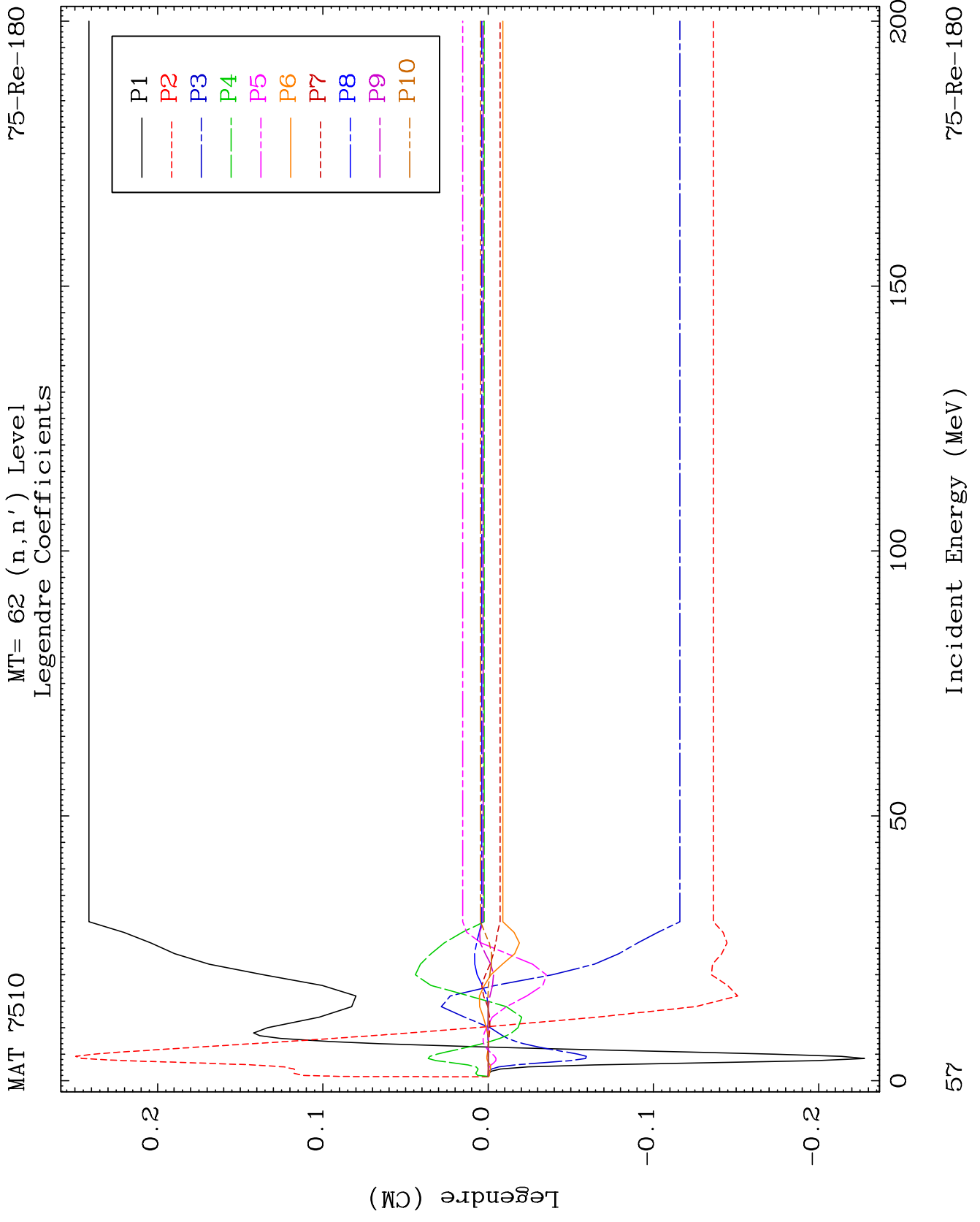


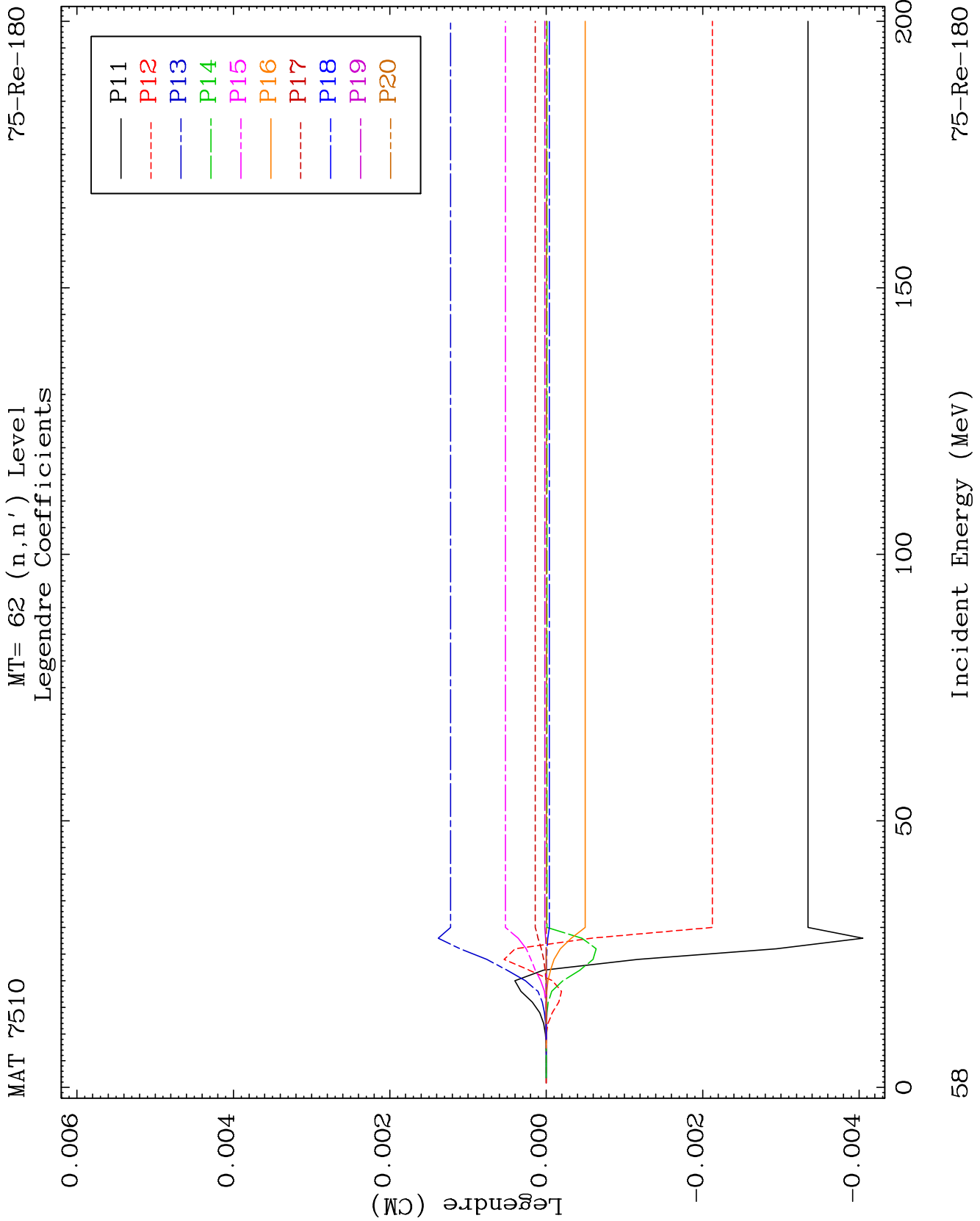


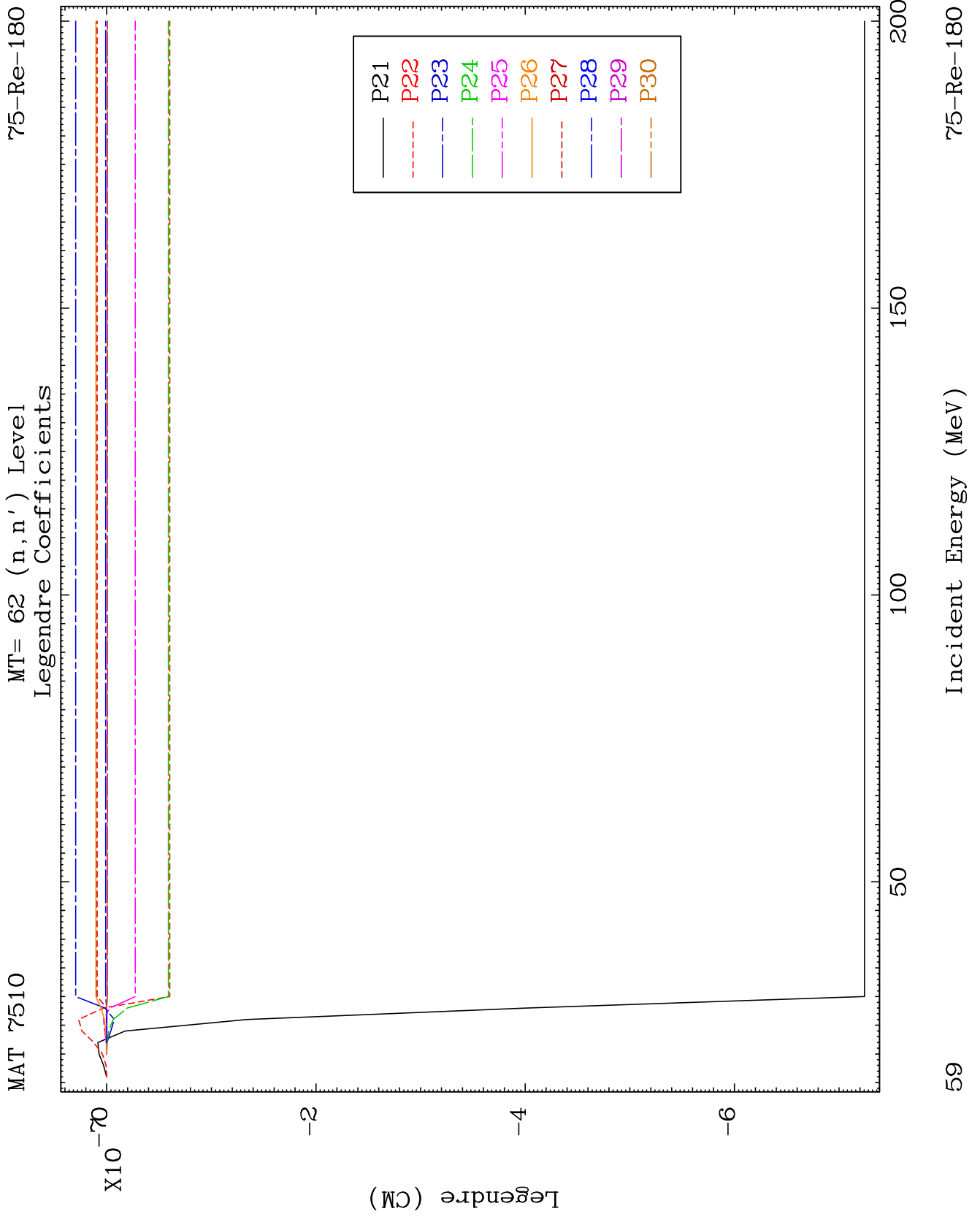


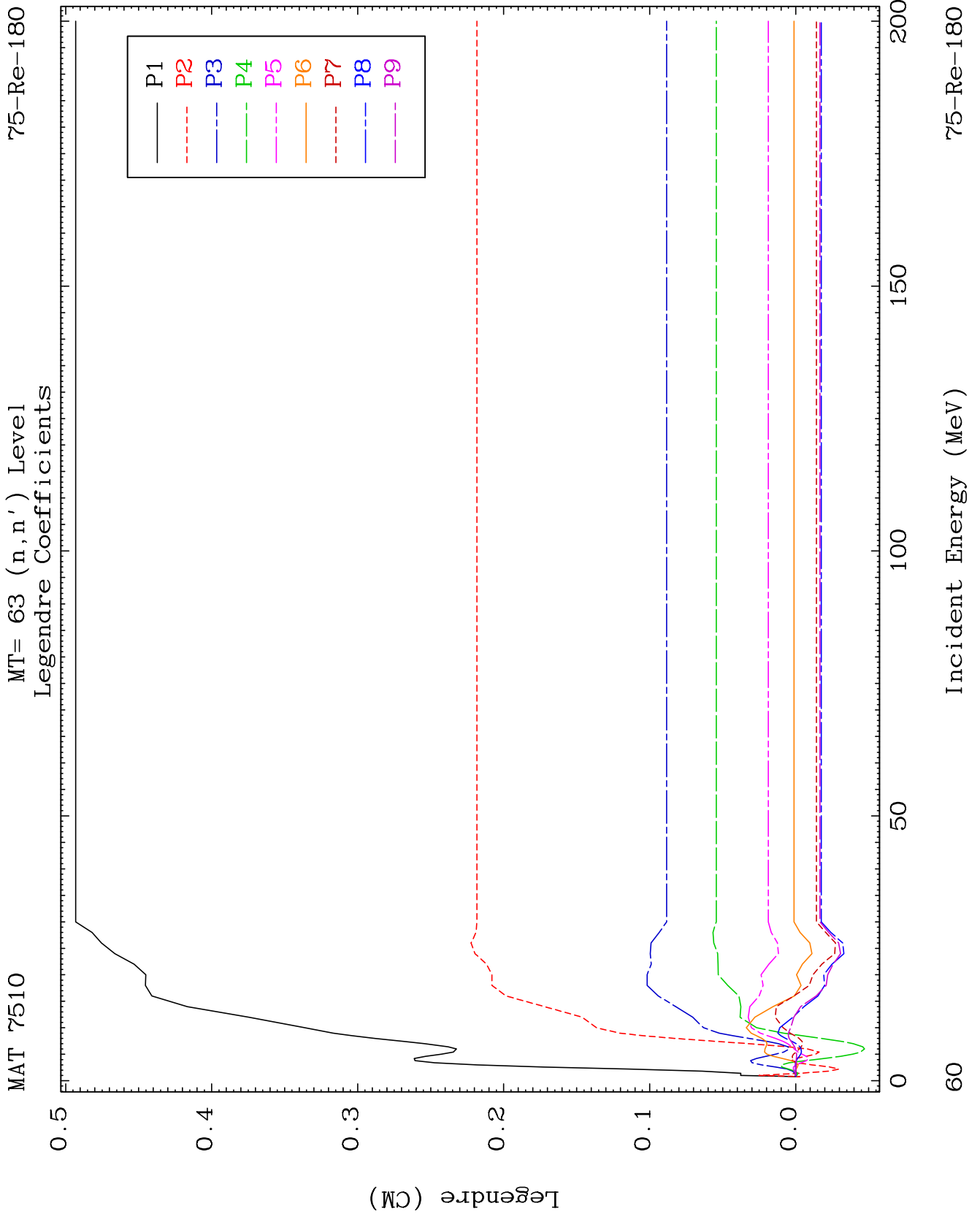






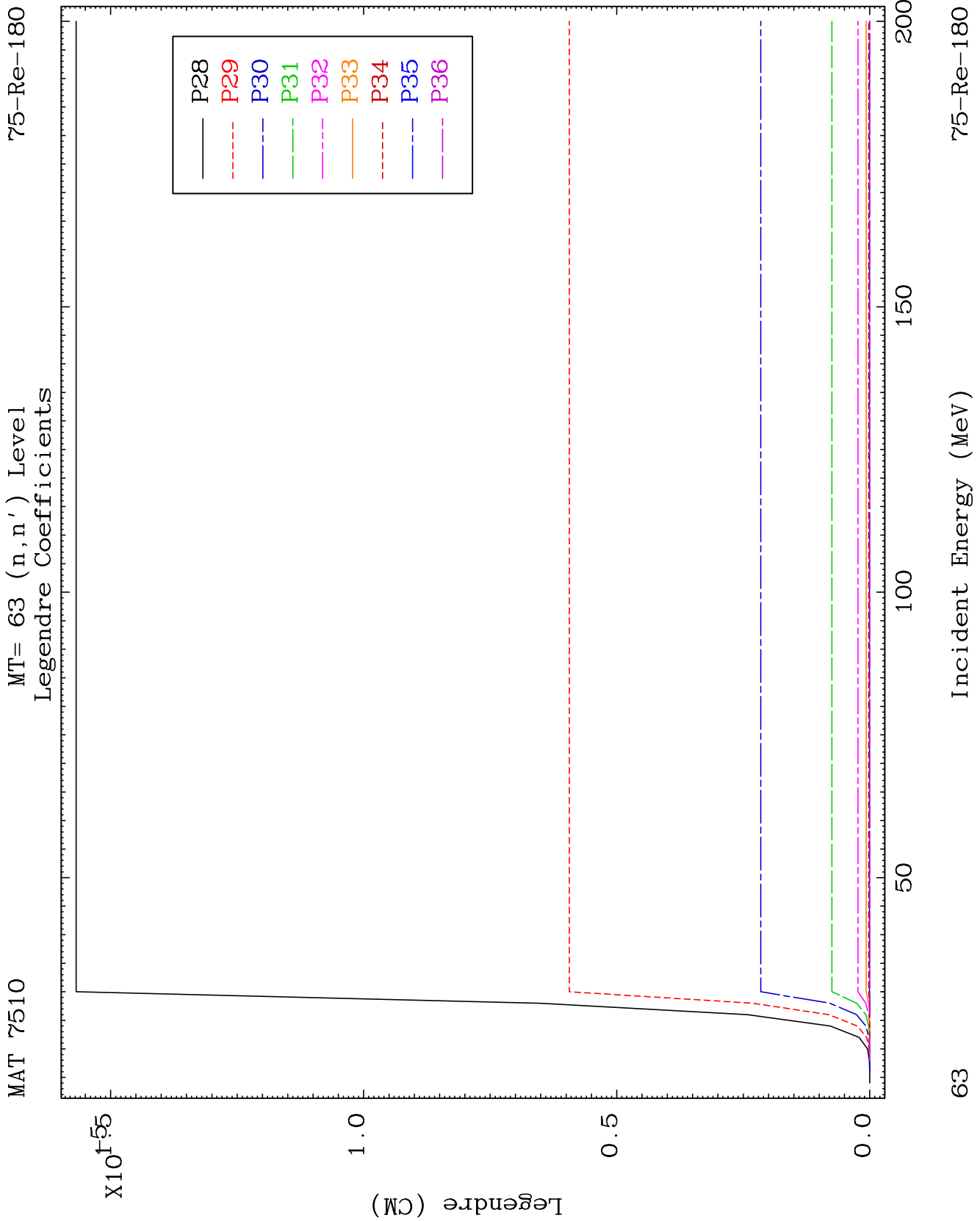




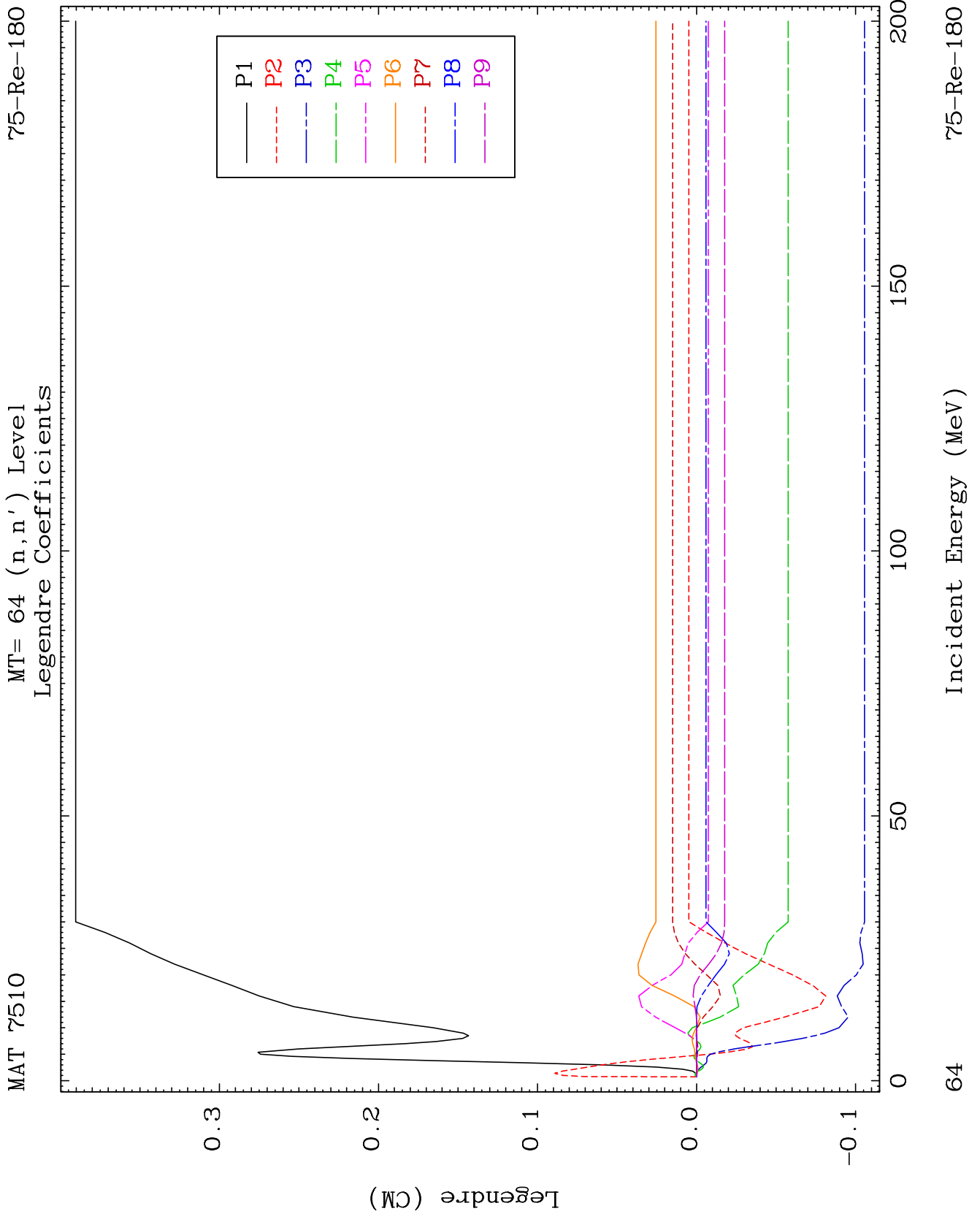


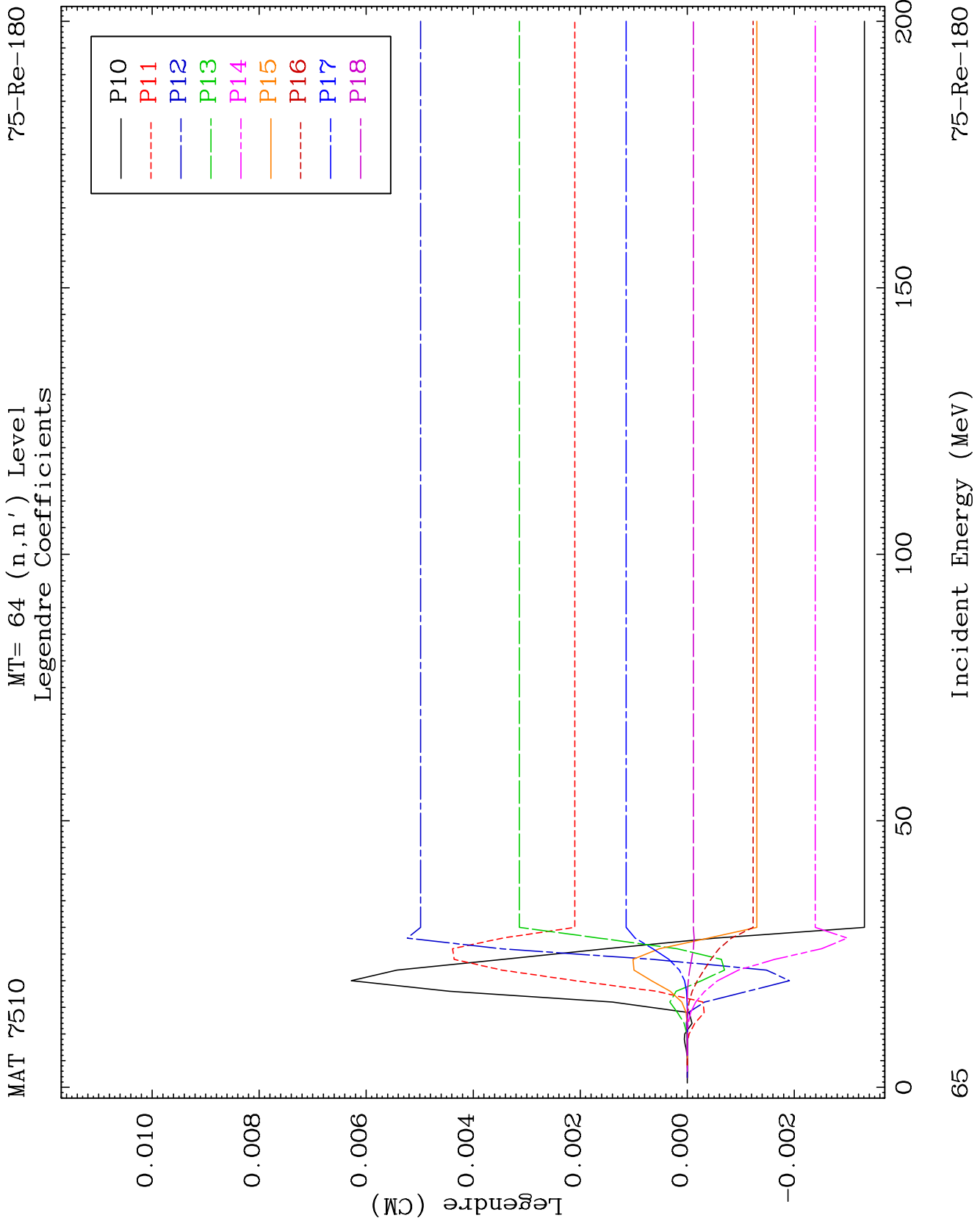


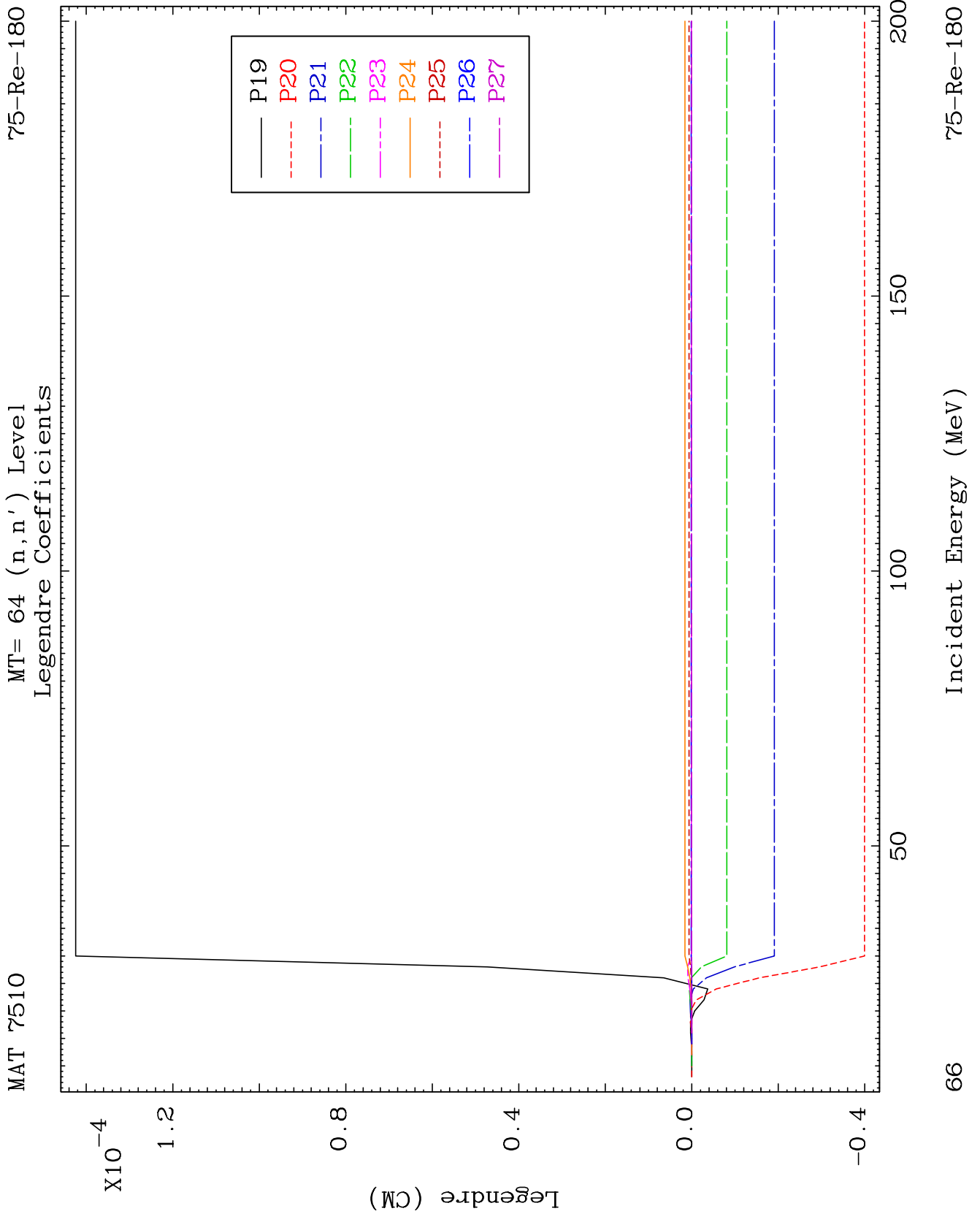








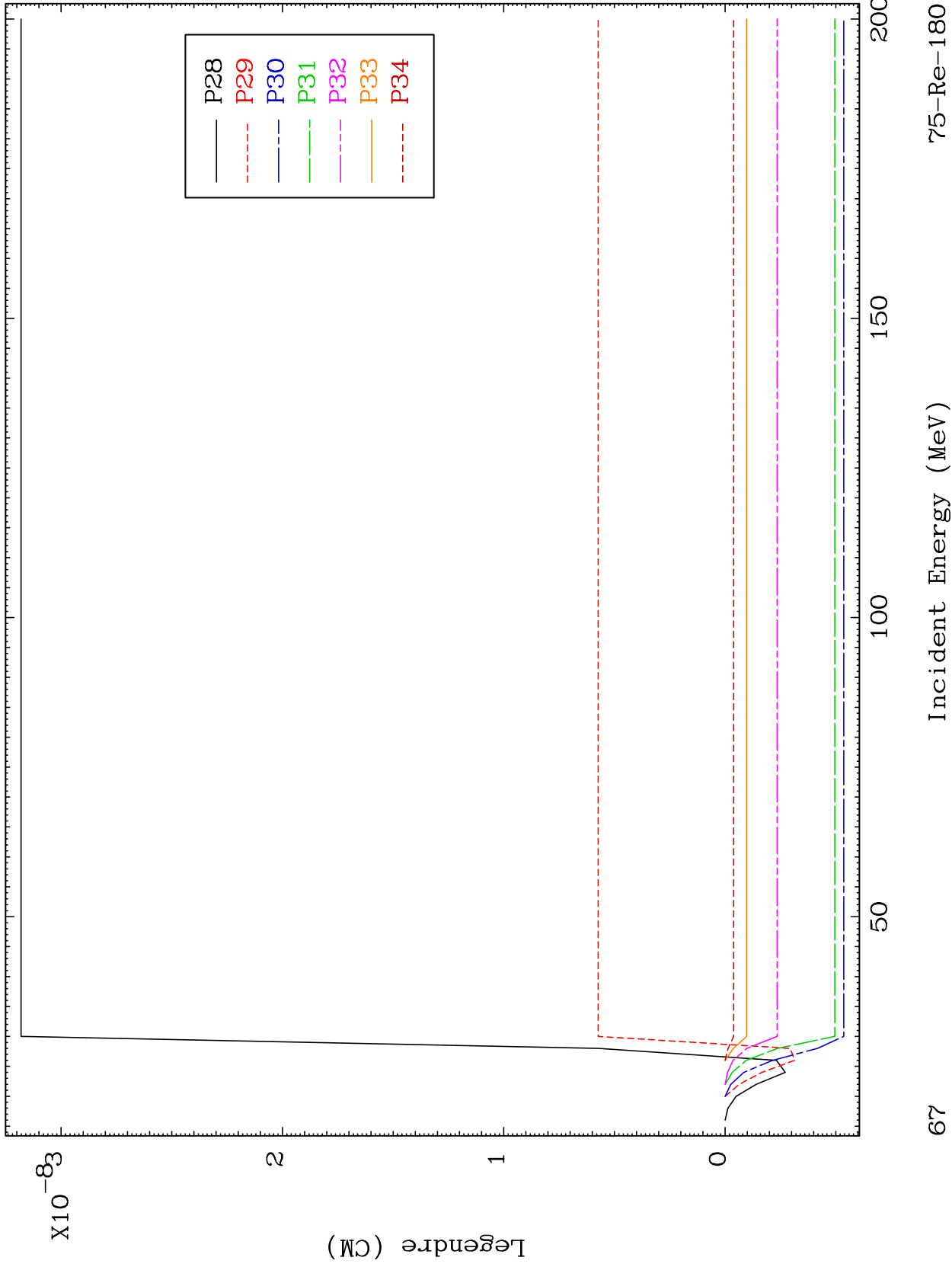




MAT 7510

MT= 64 (n,n') Level  
Legendre Coefficients

75-Re-180

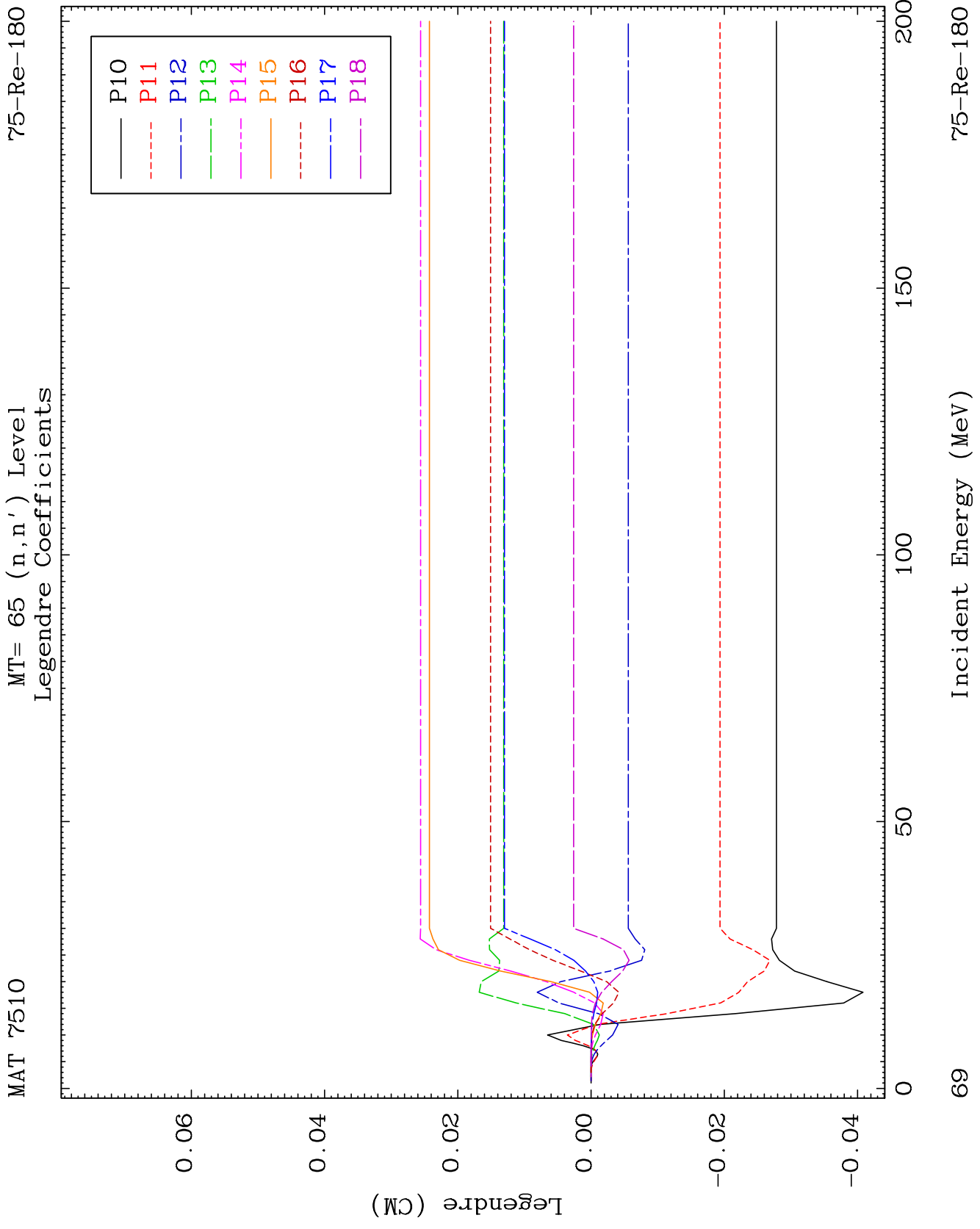


67

Incident Energy (MeV)

75-Re-180

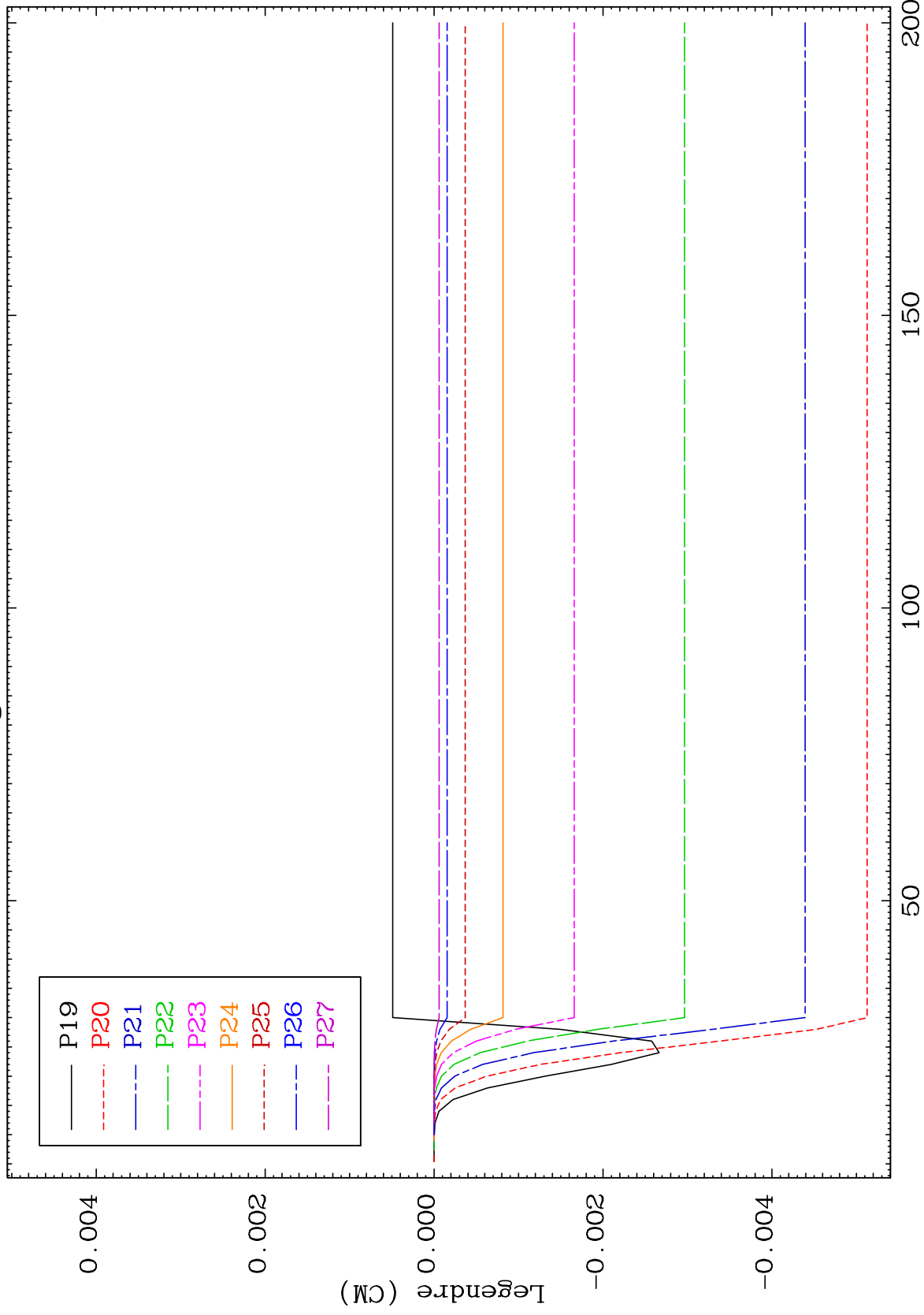




MAT 7510

MT= 65 (n,n') Level  
Legendre Coefficients

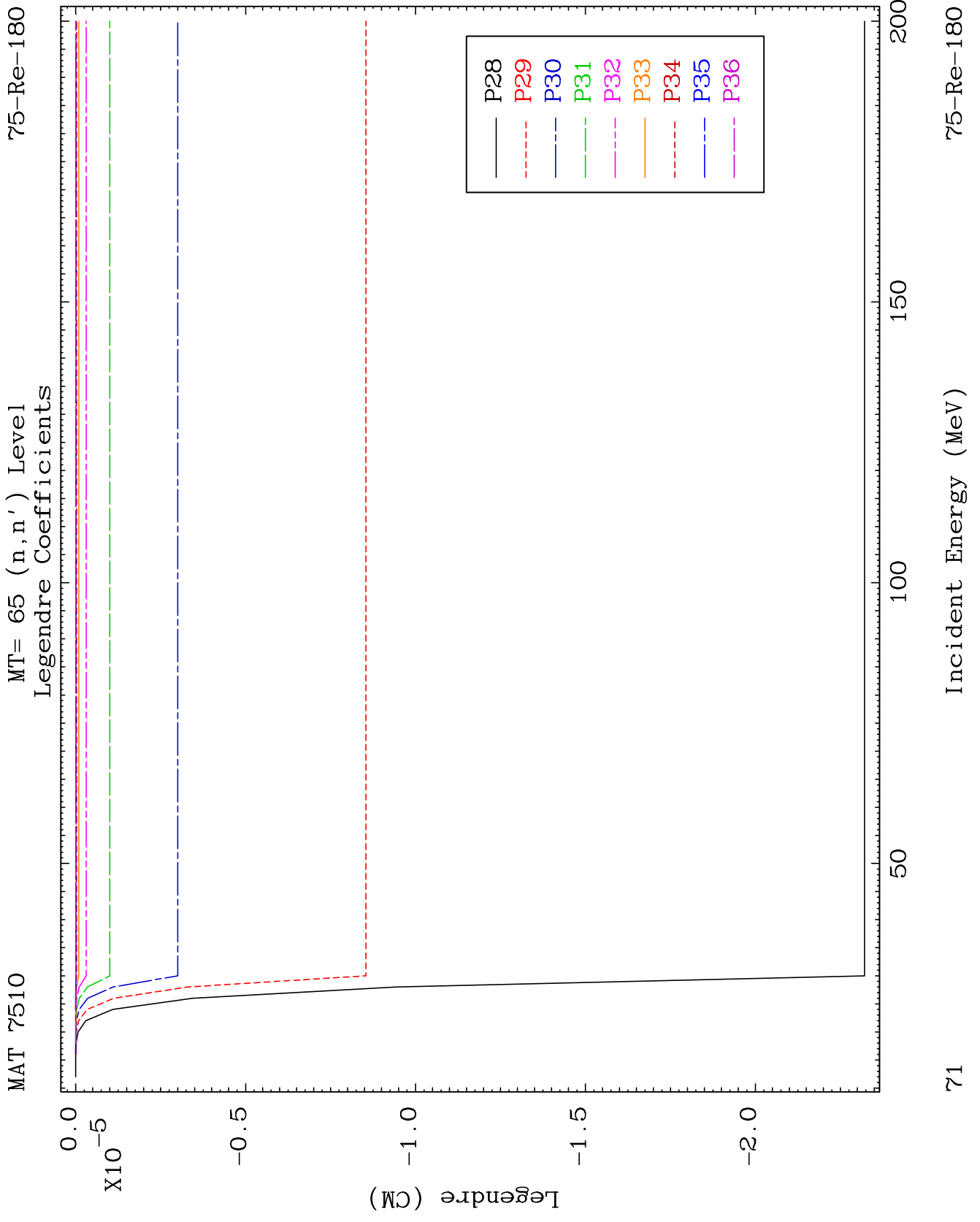
75-Re-180



70

Incident Energy (MeV)

75-Re-180



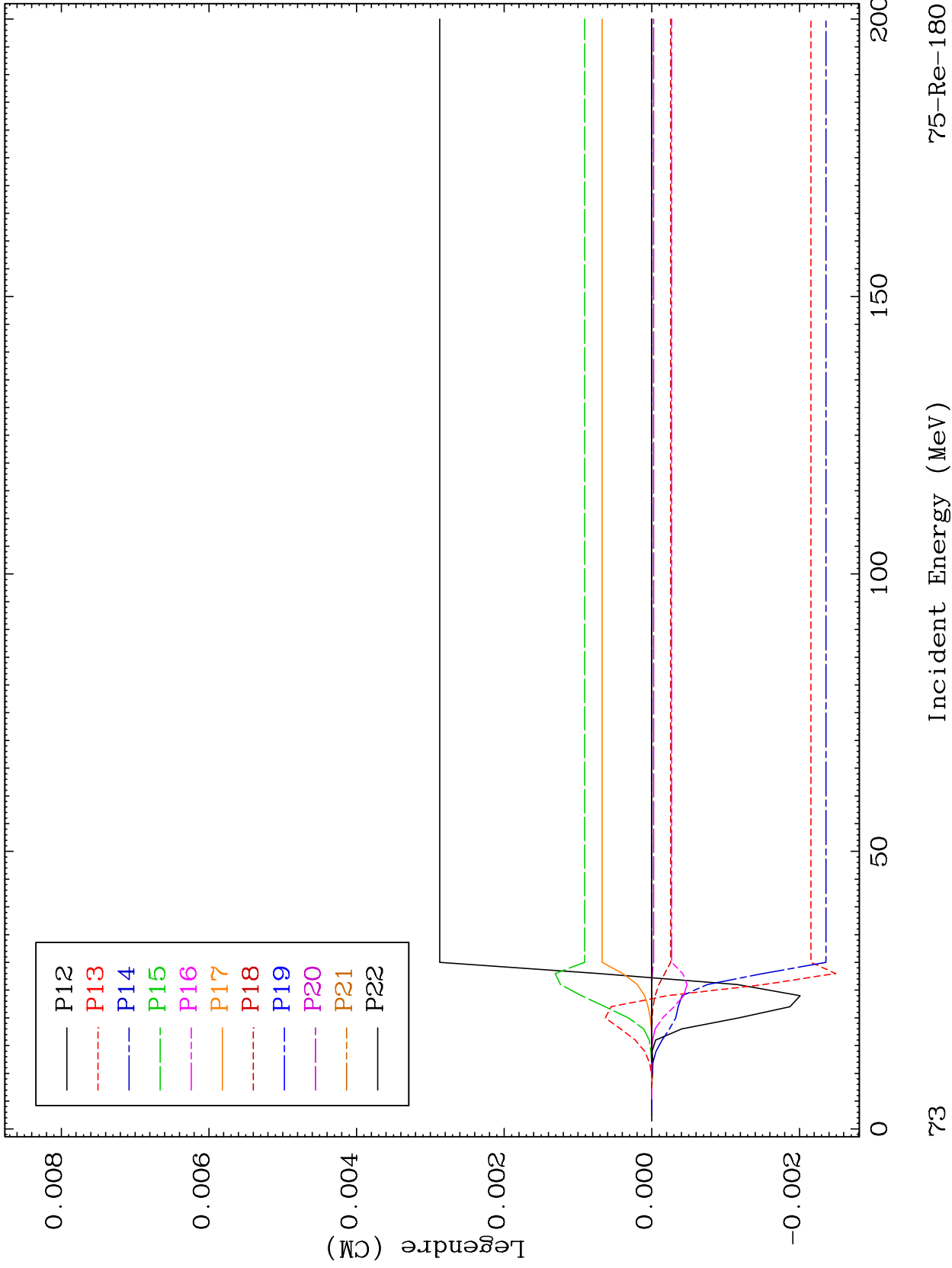




MAT 7510

MT= 66 (n,n') Level  
Legendre Coefficients

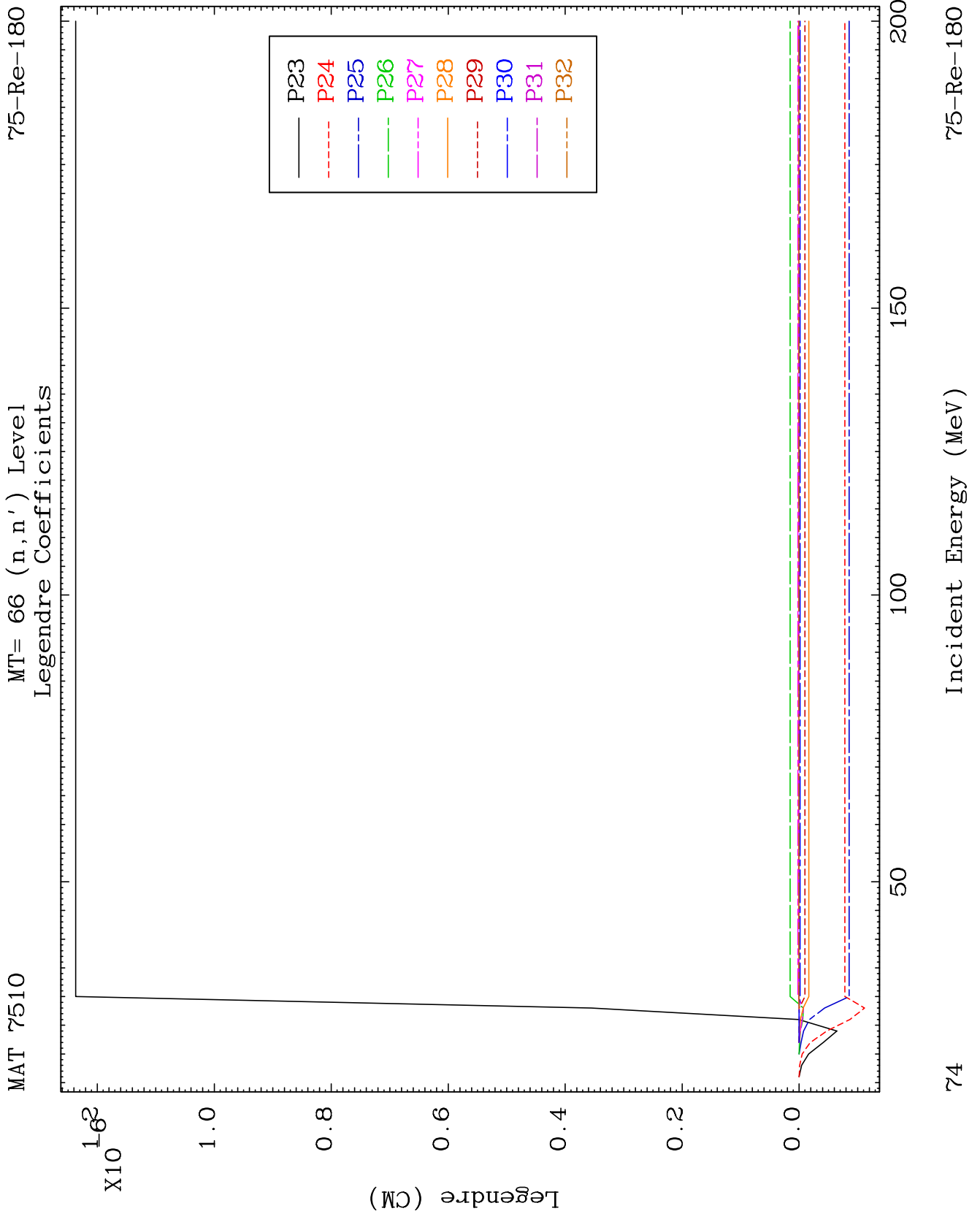
75-Re-180

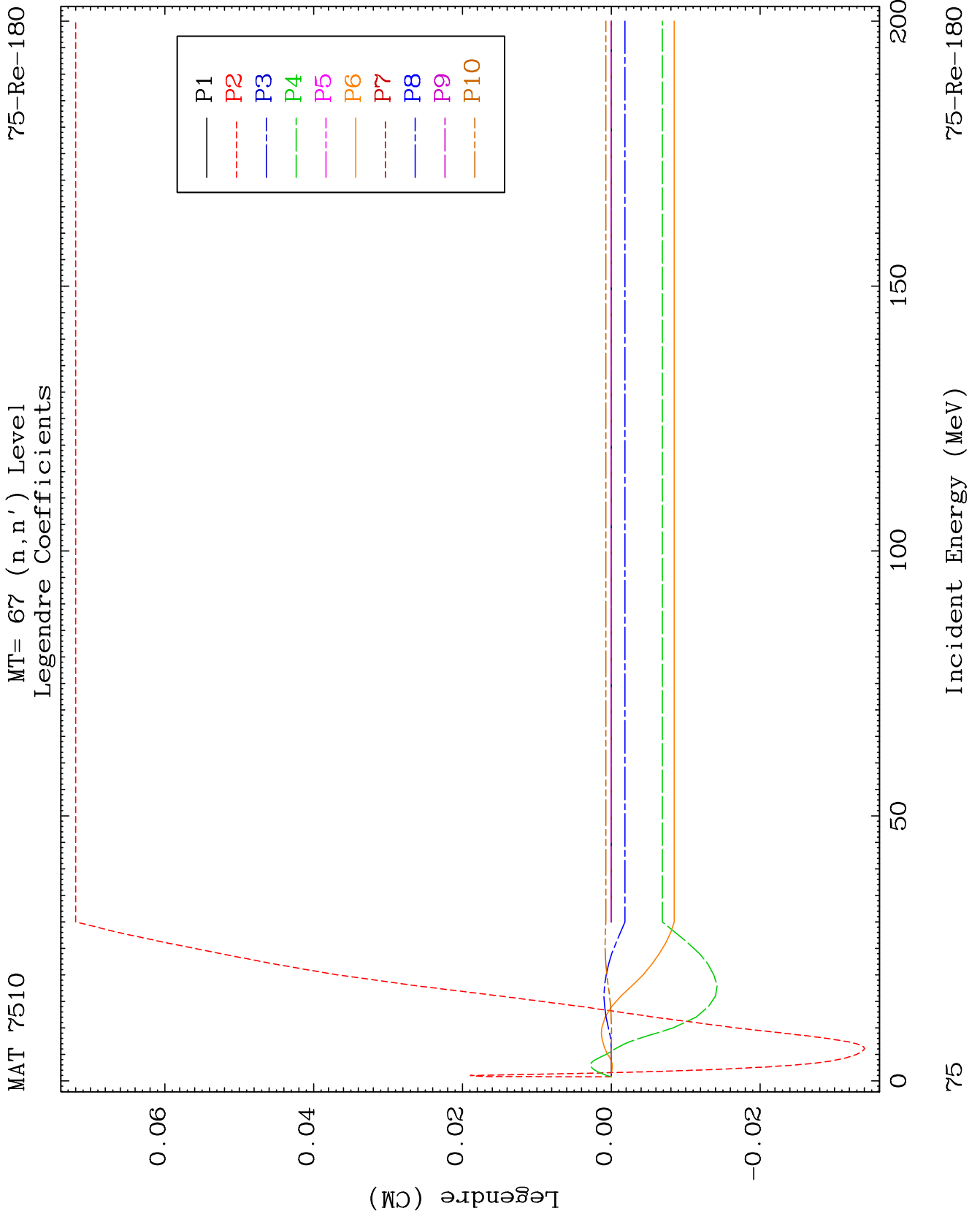


73

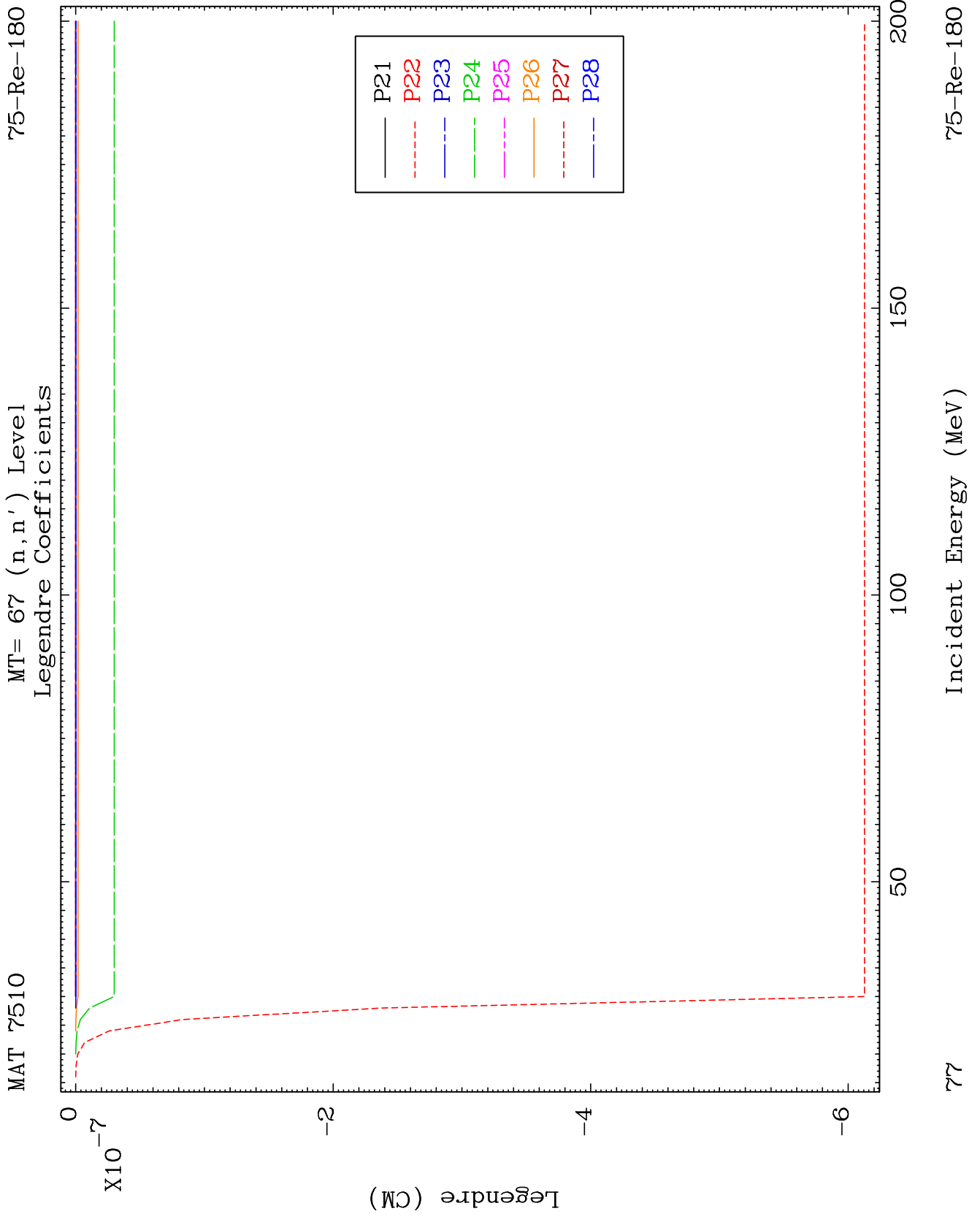
Incident Energy (MeV)

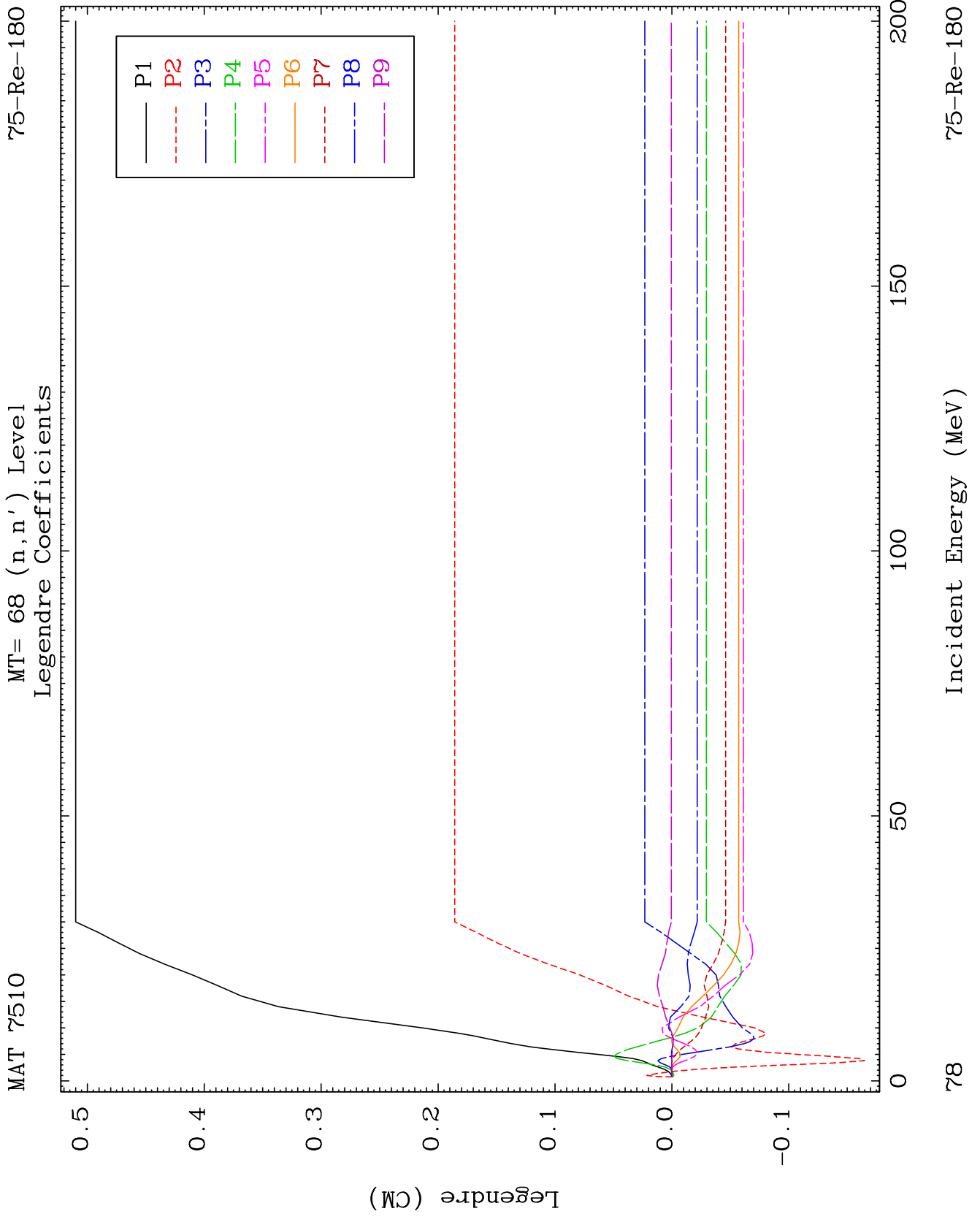
75-Re-180





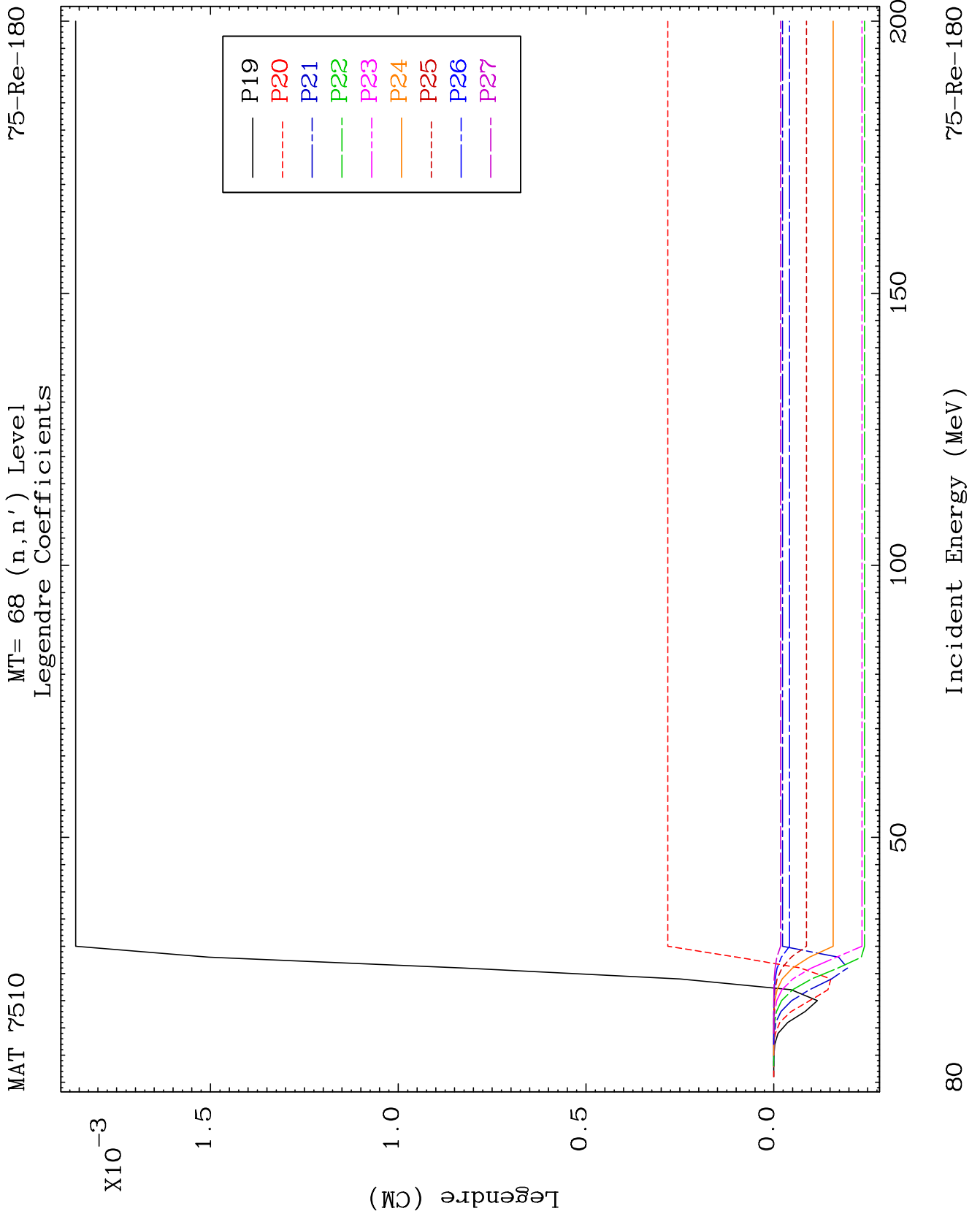


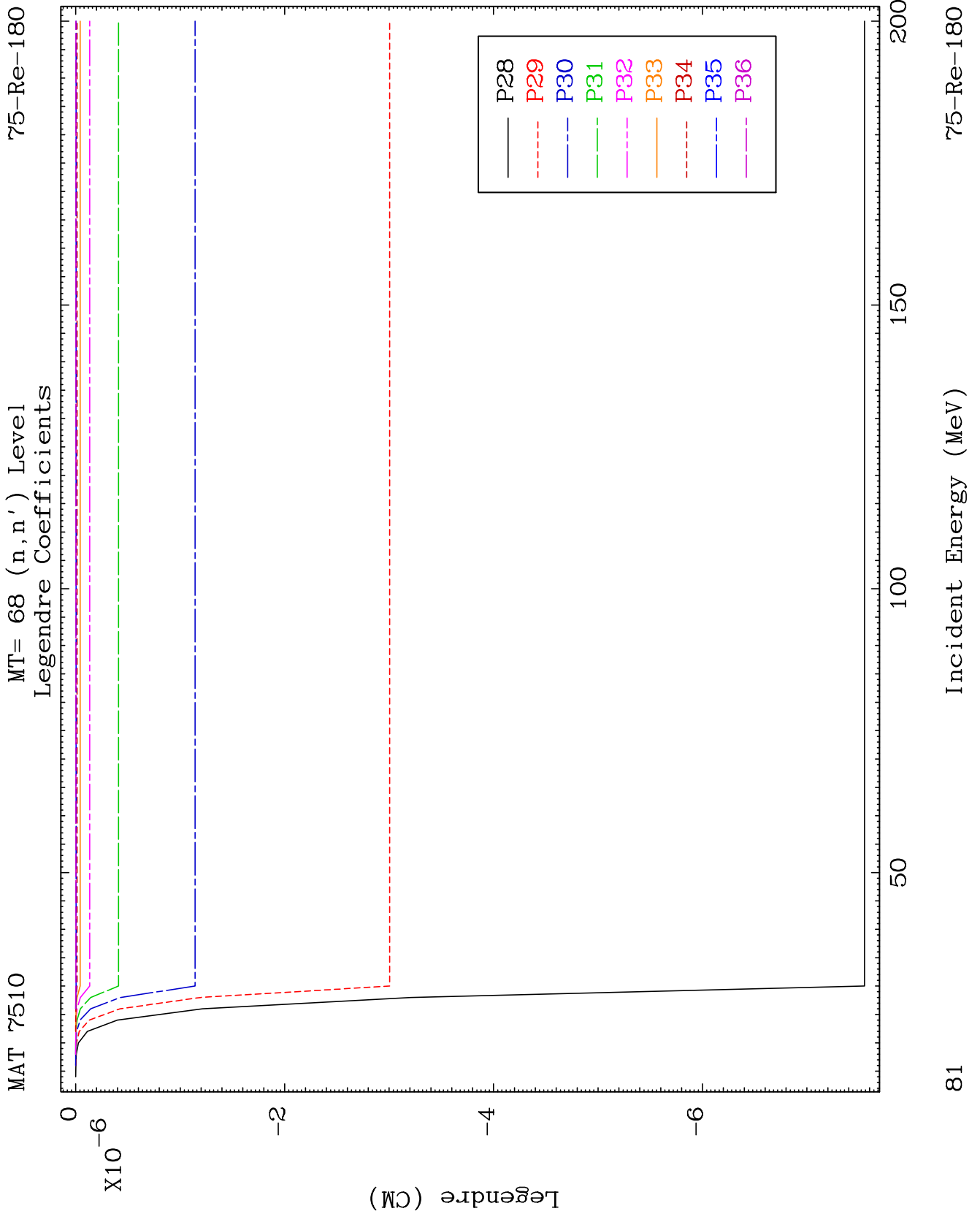


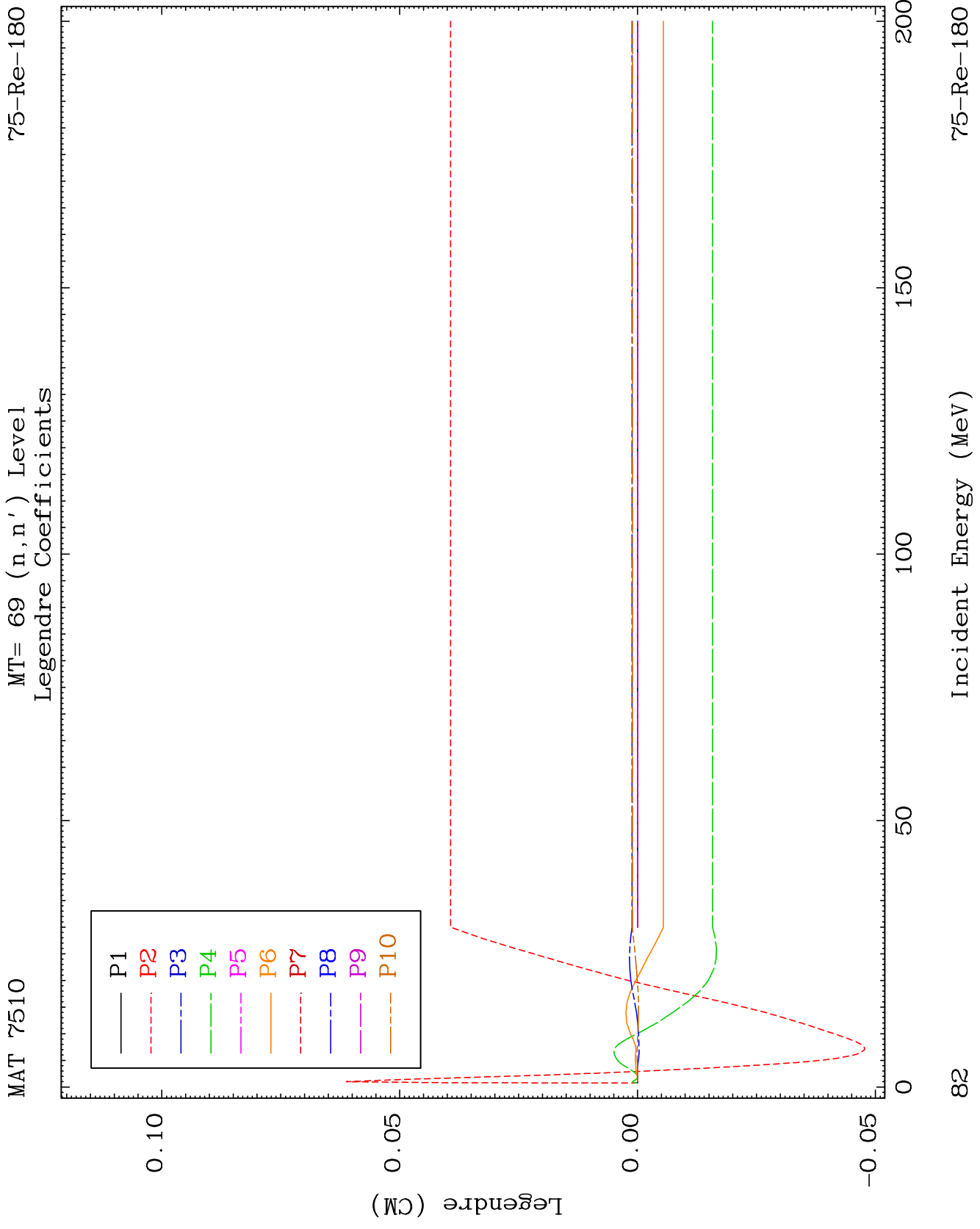


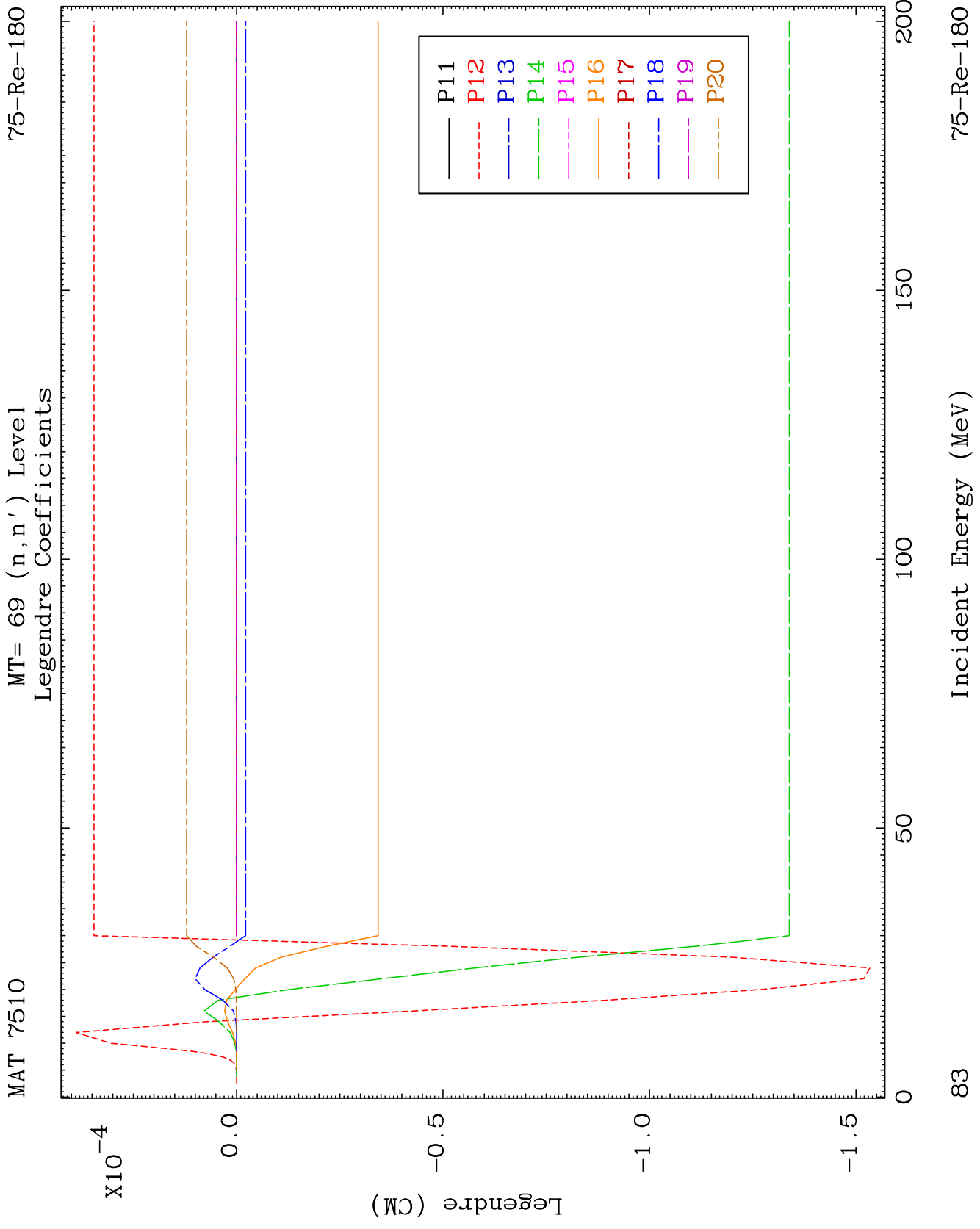




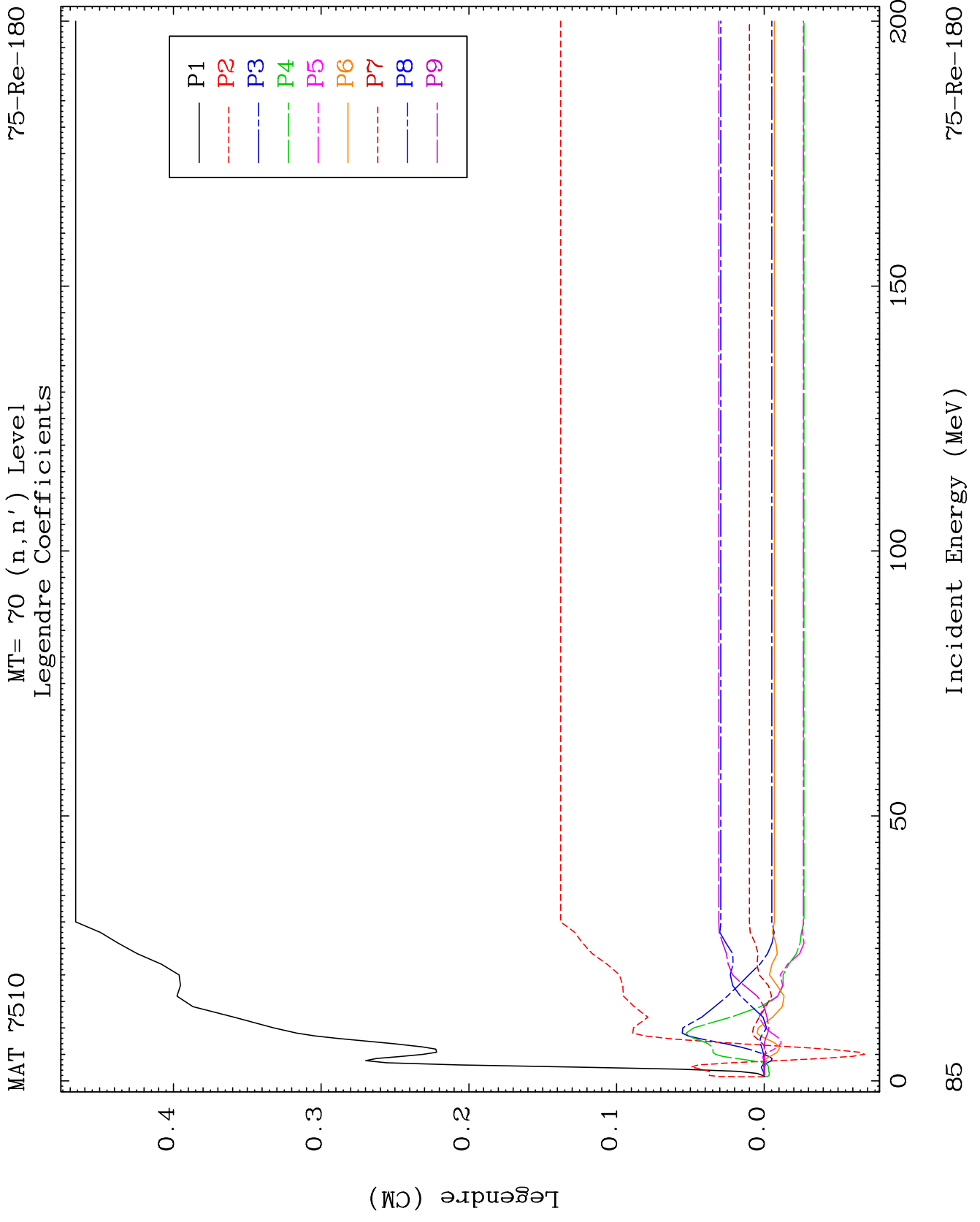


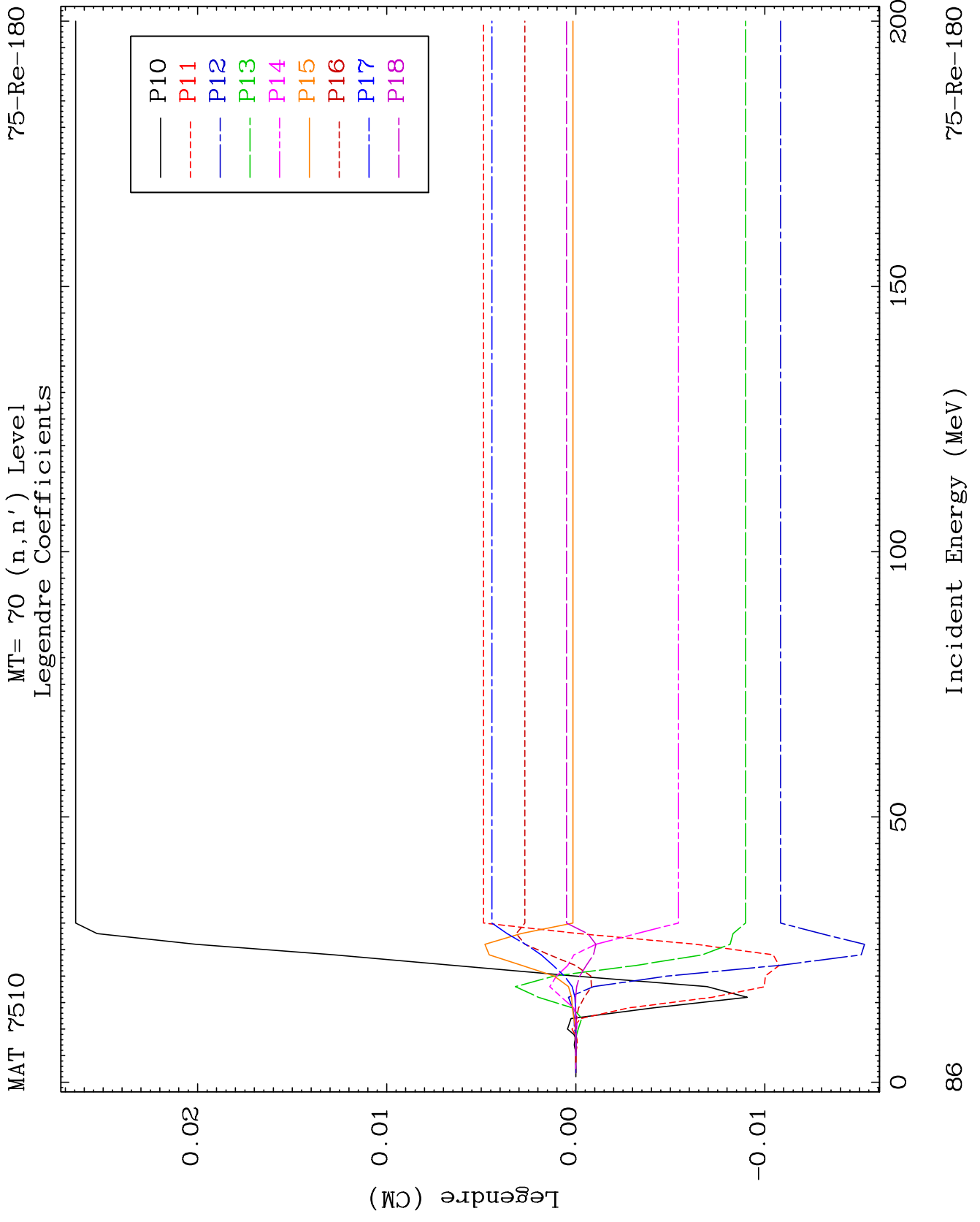


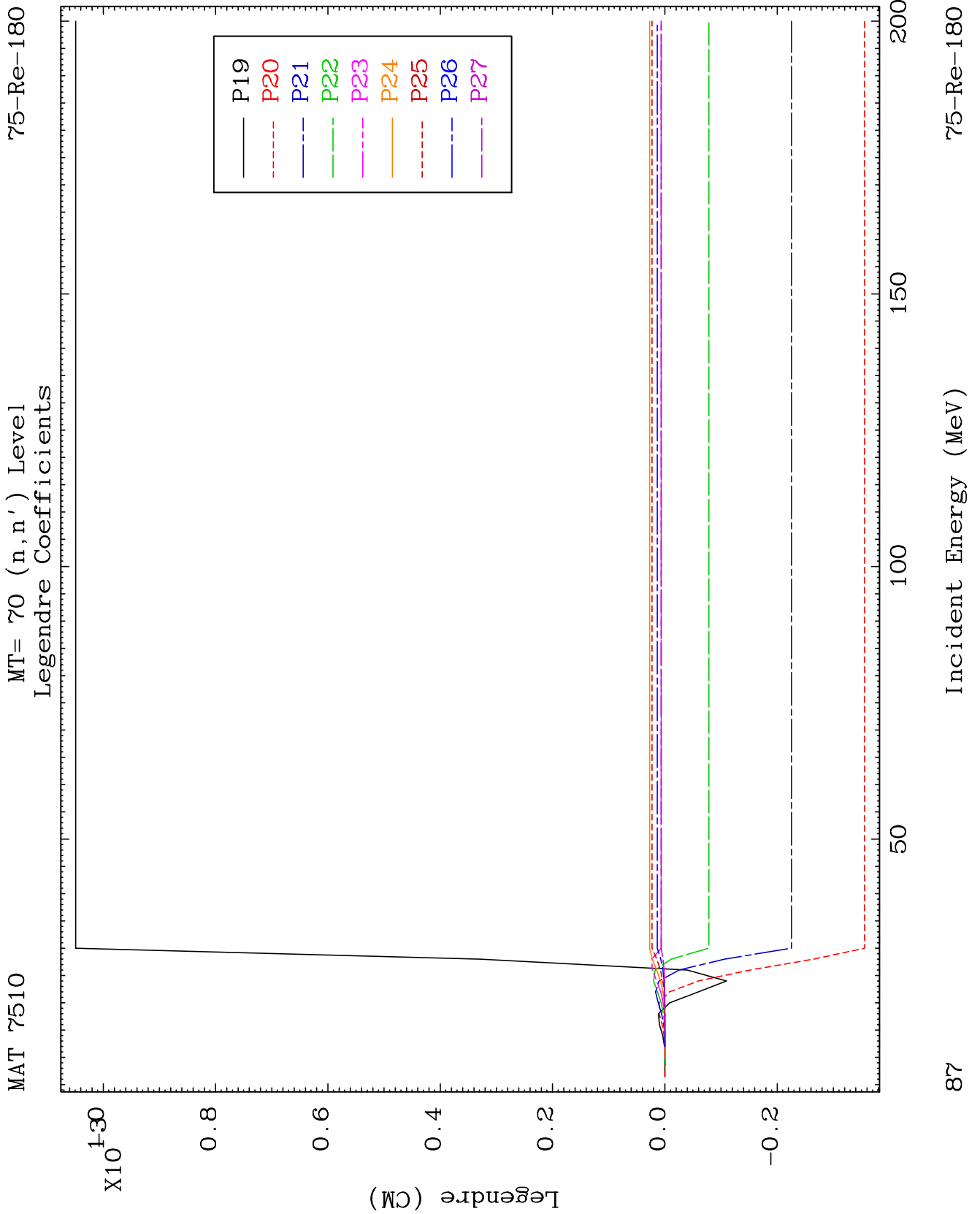




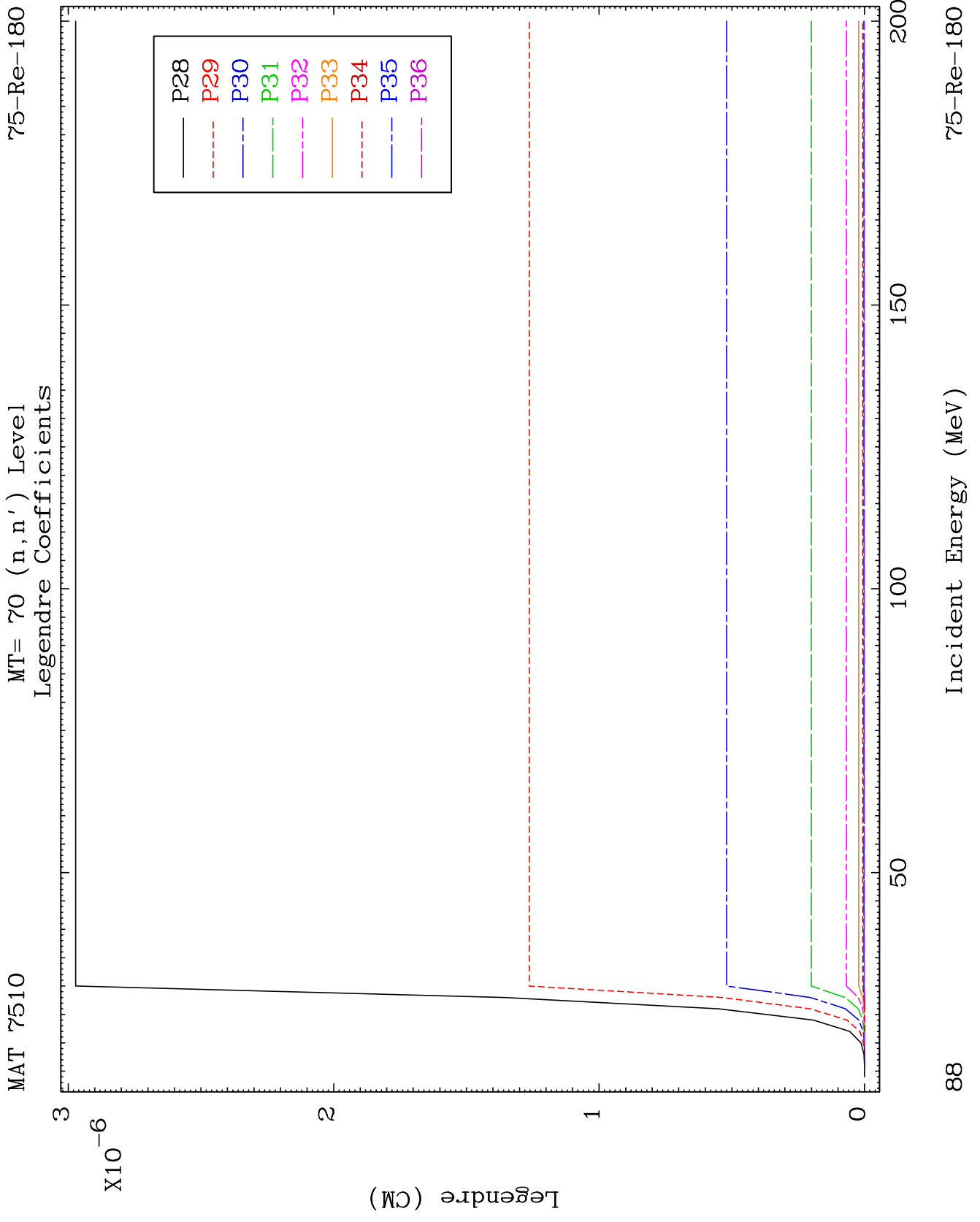


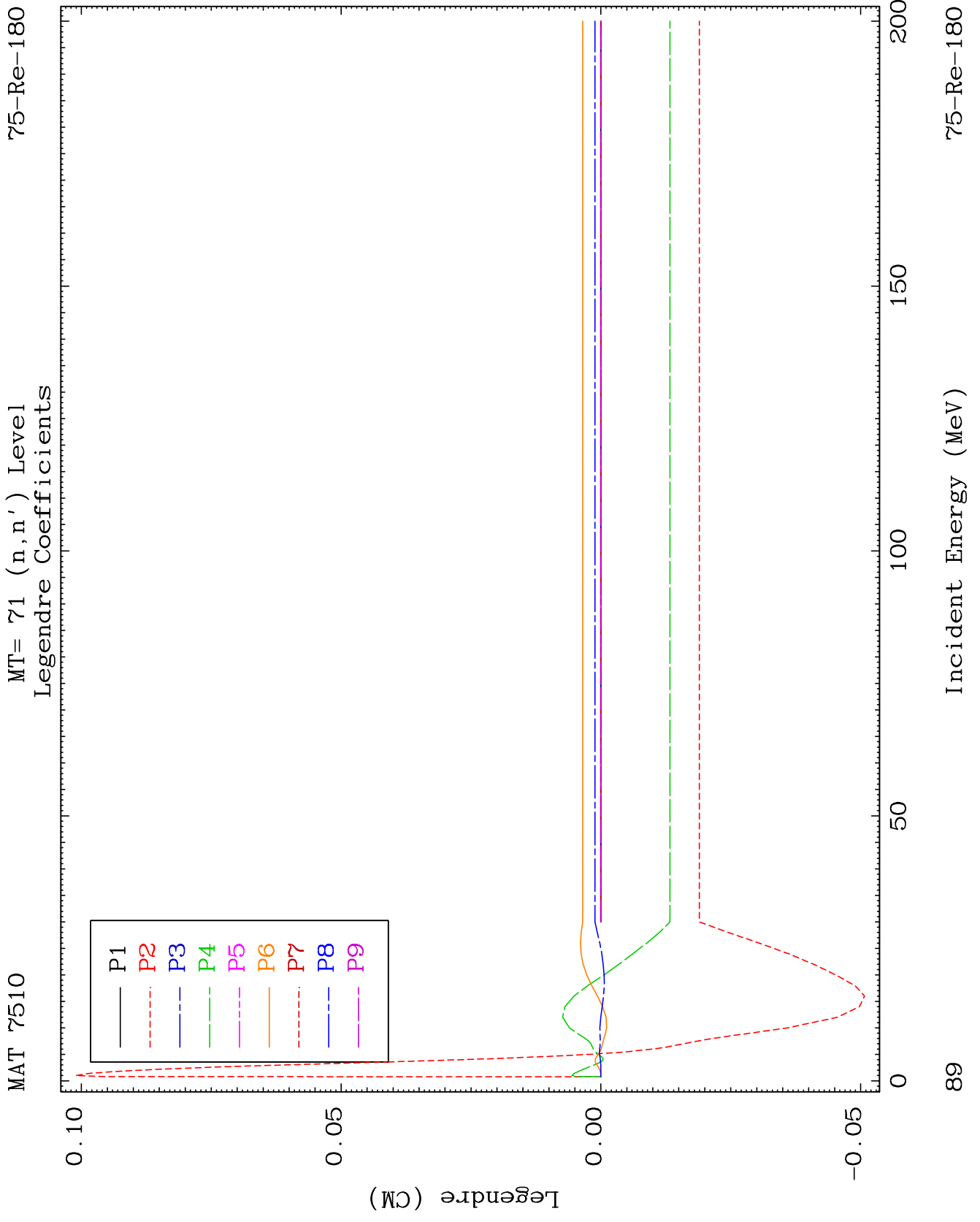


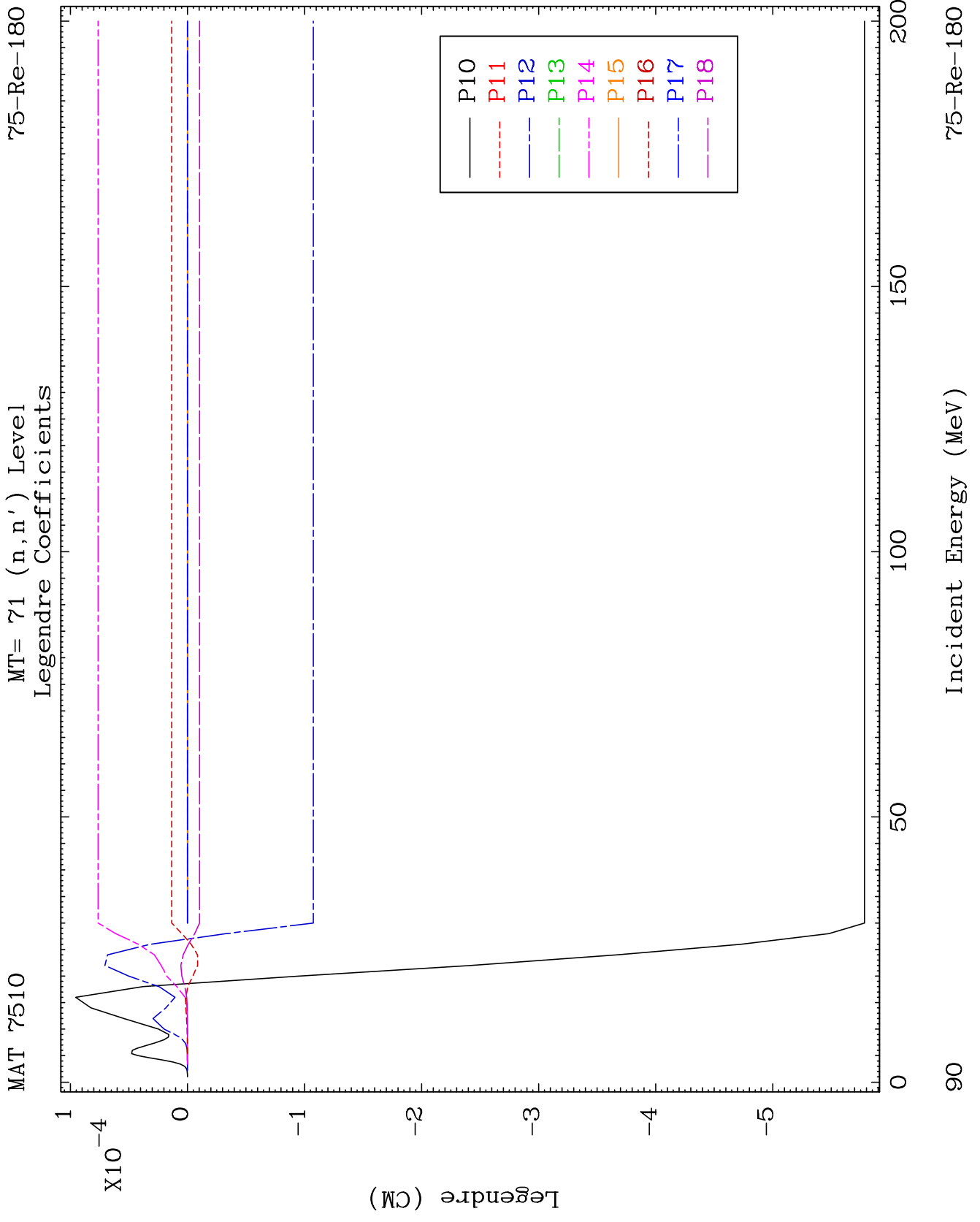


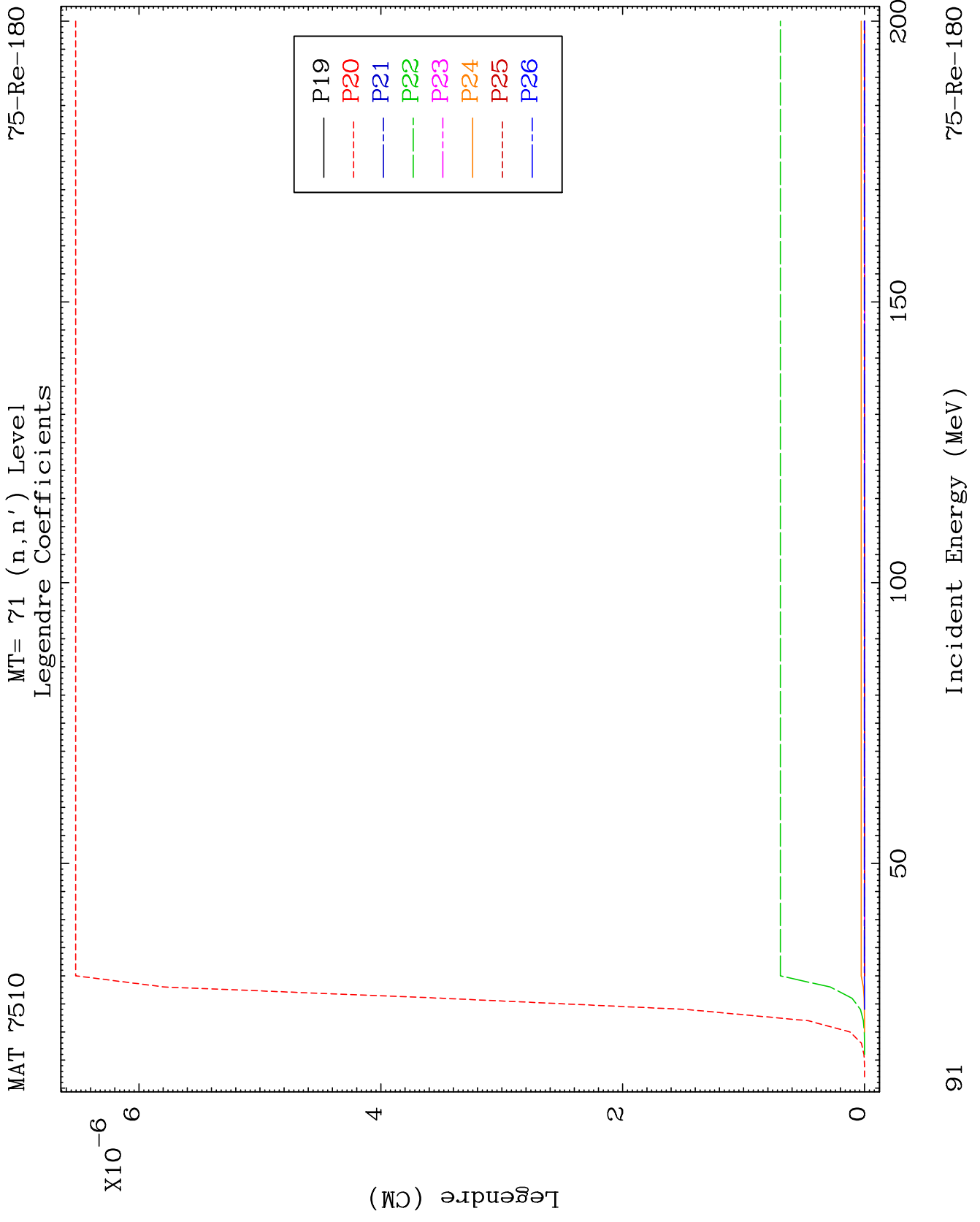




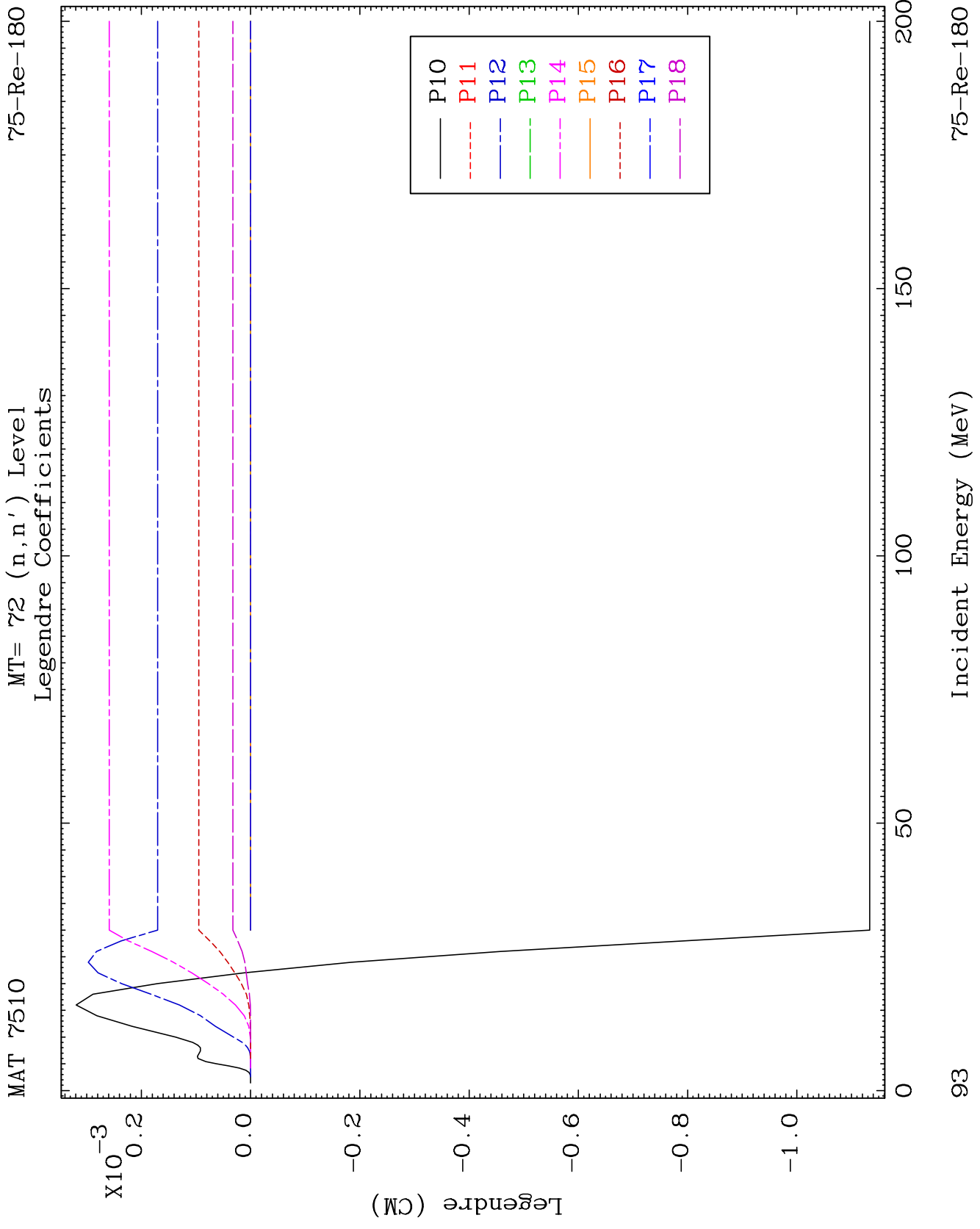


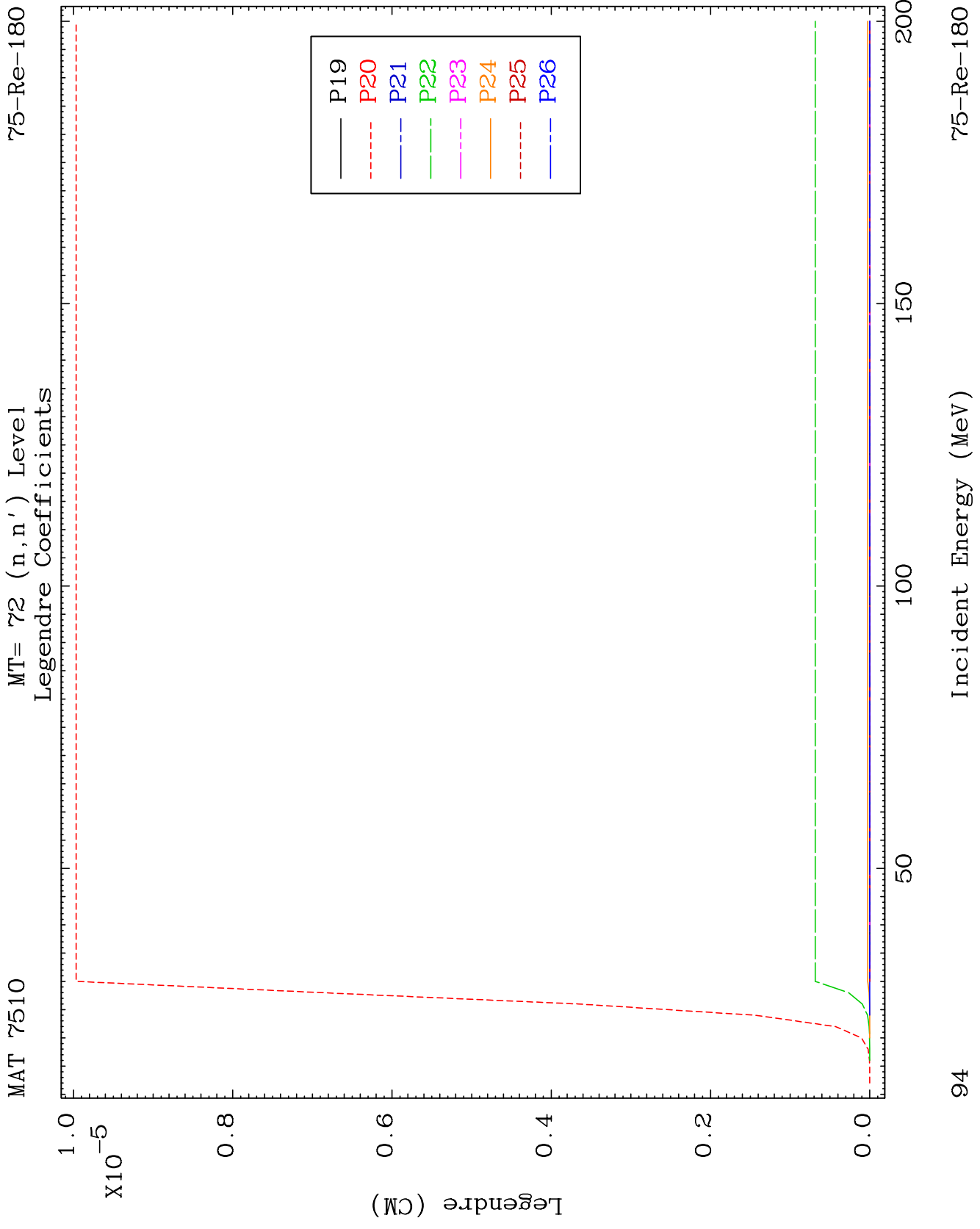


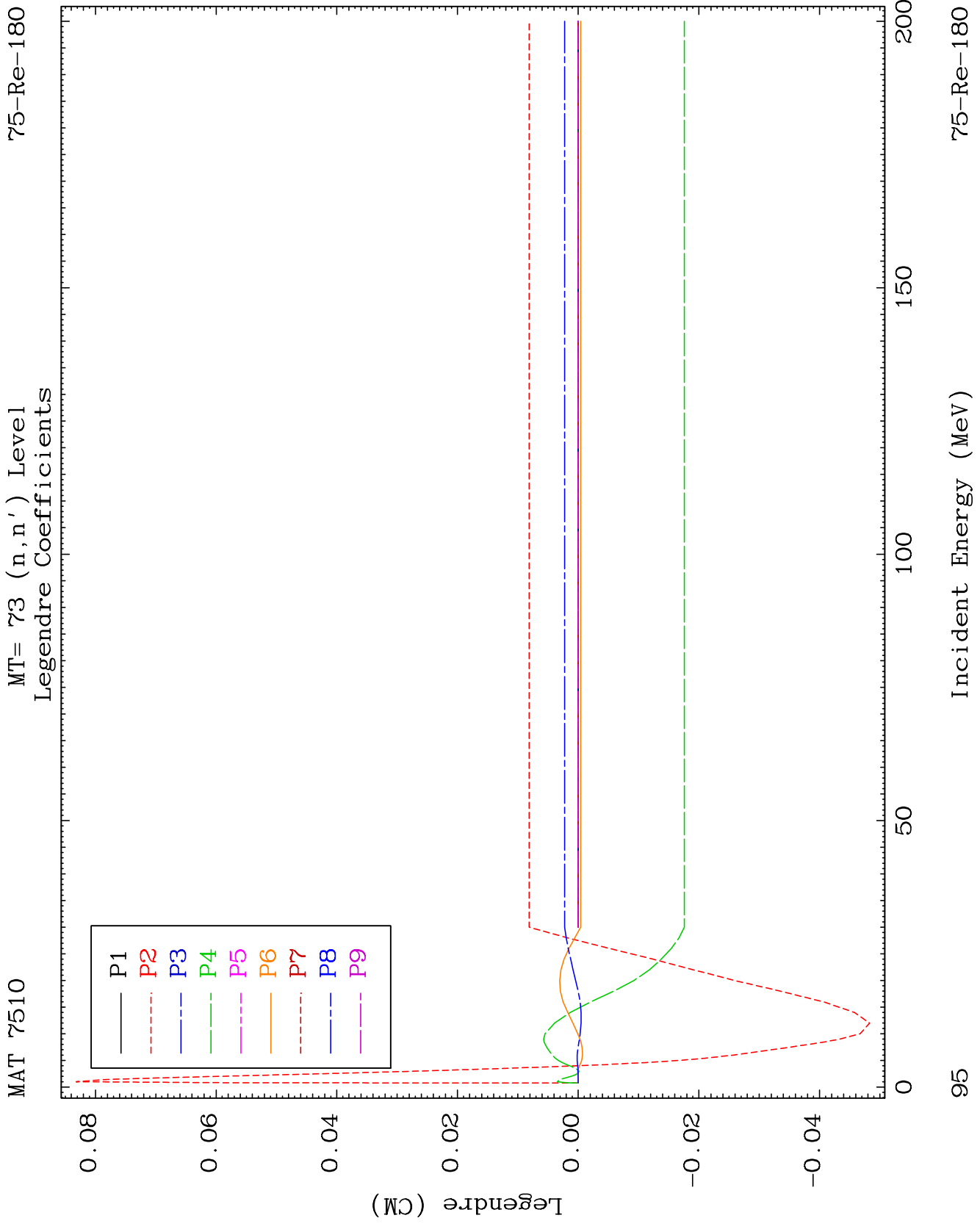










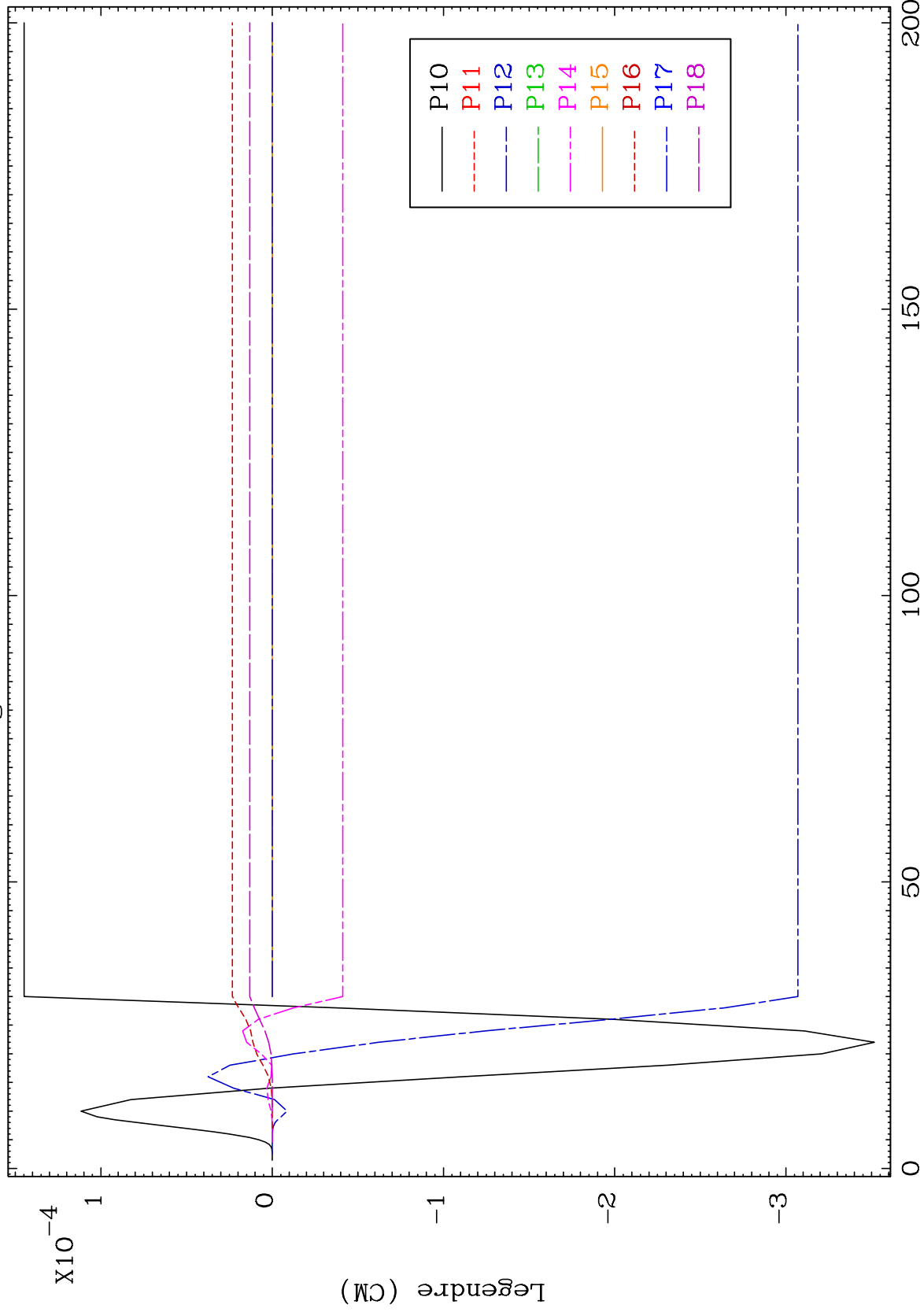




MAT 7510

MT= 73 (n,n') Level  
Legendre Coefficients

75-Re-180

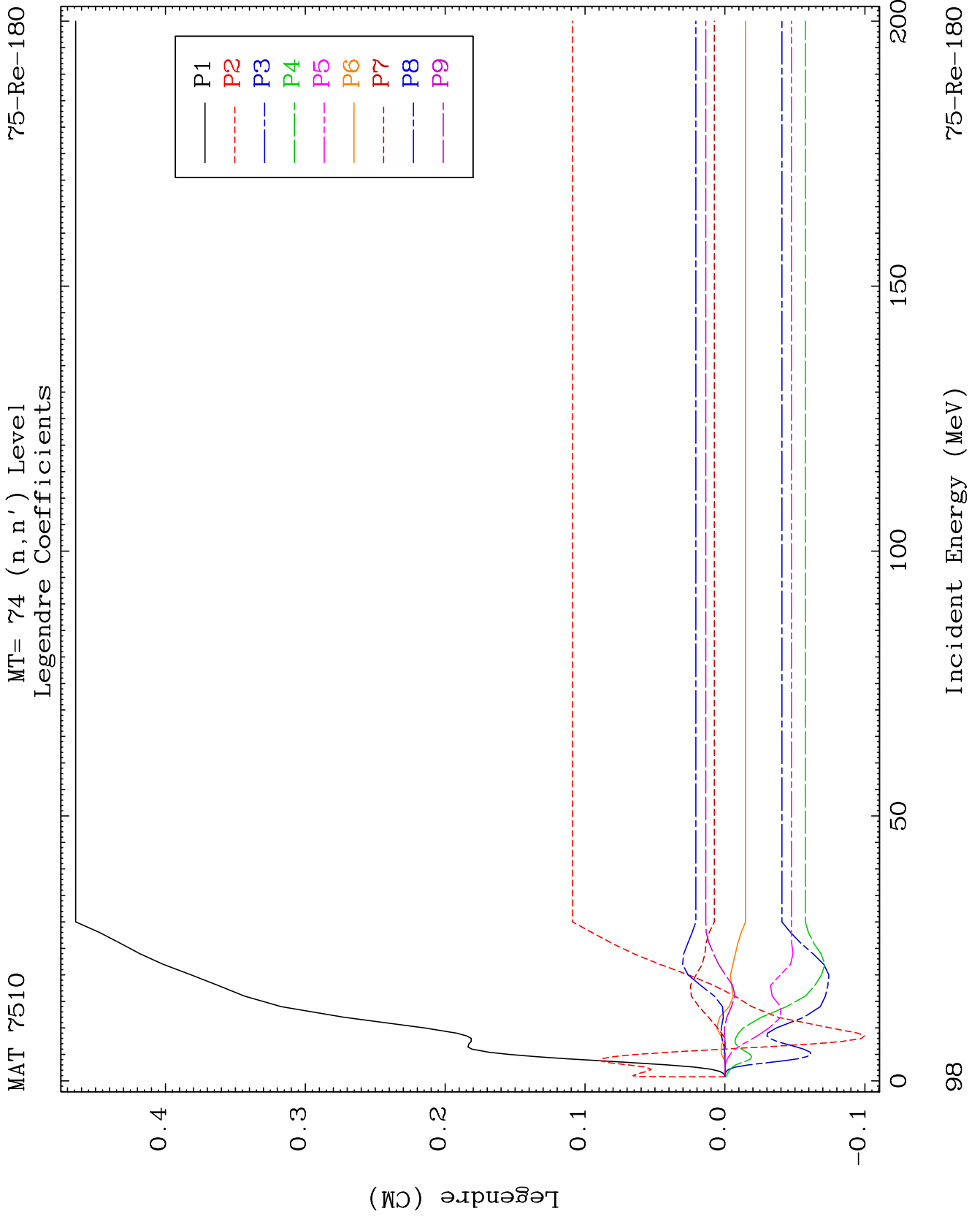


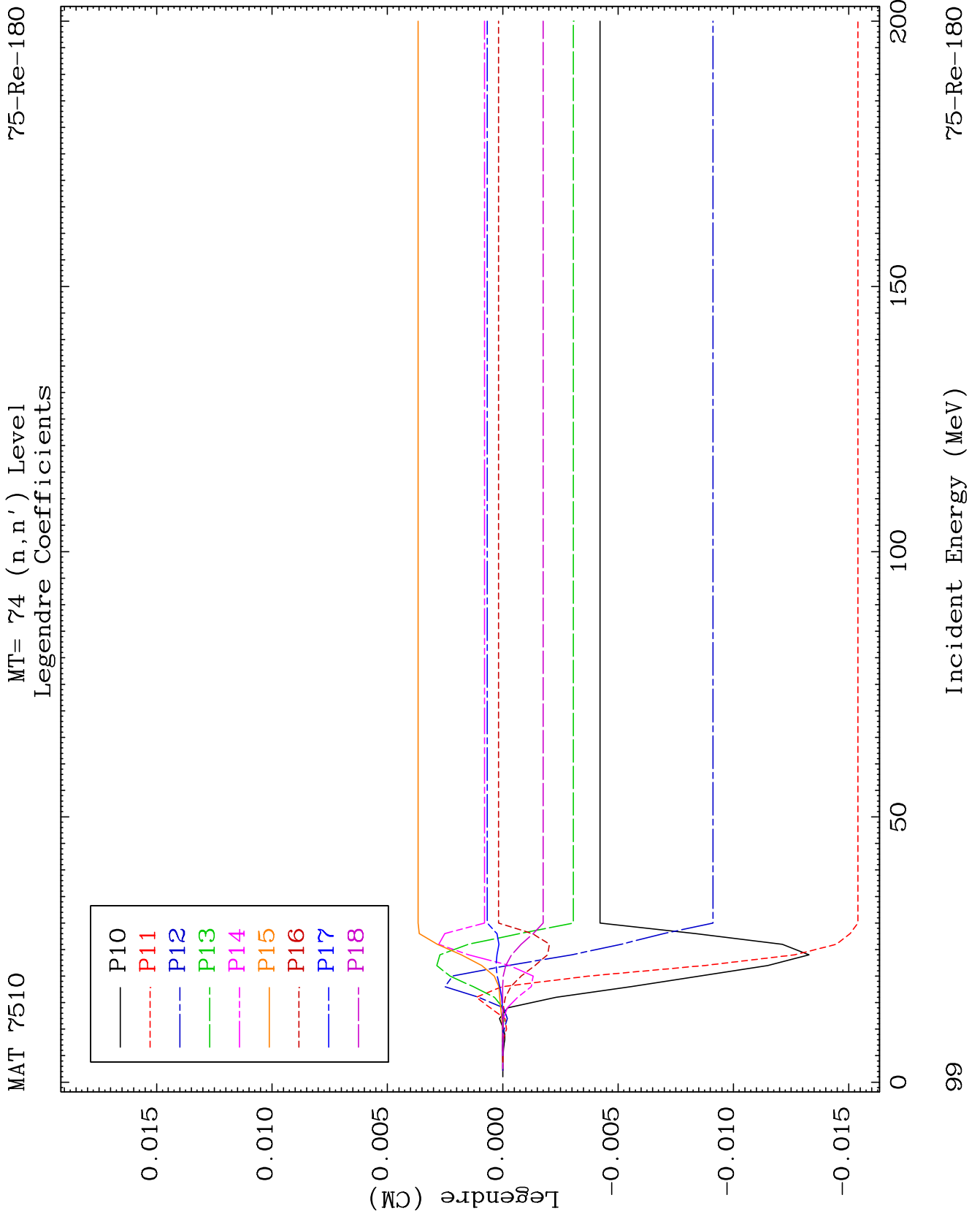
96

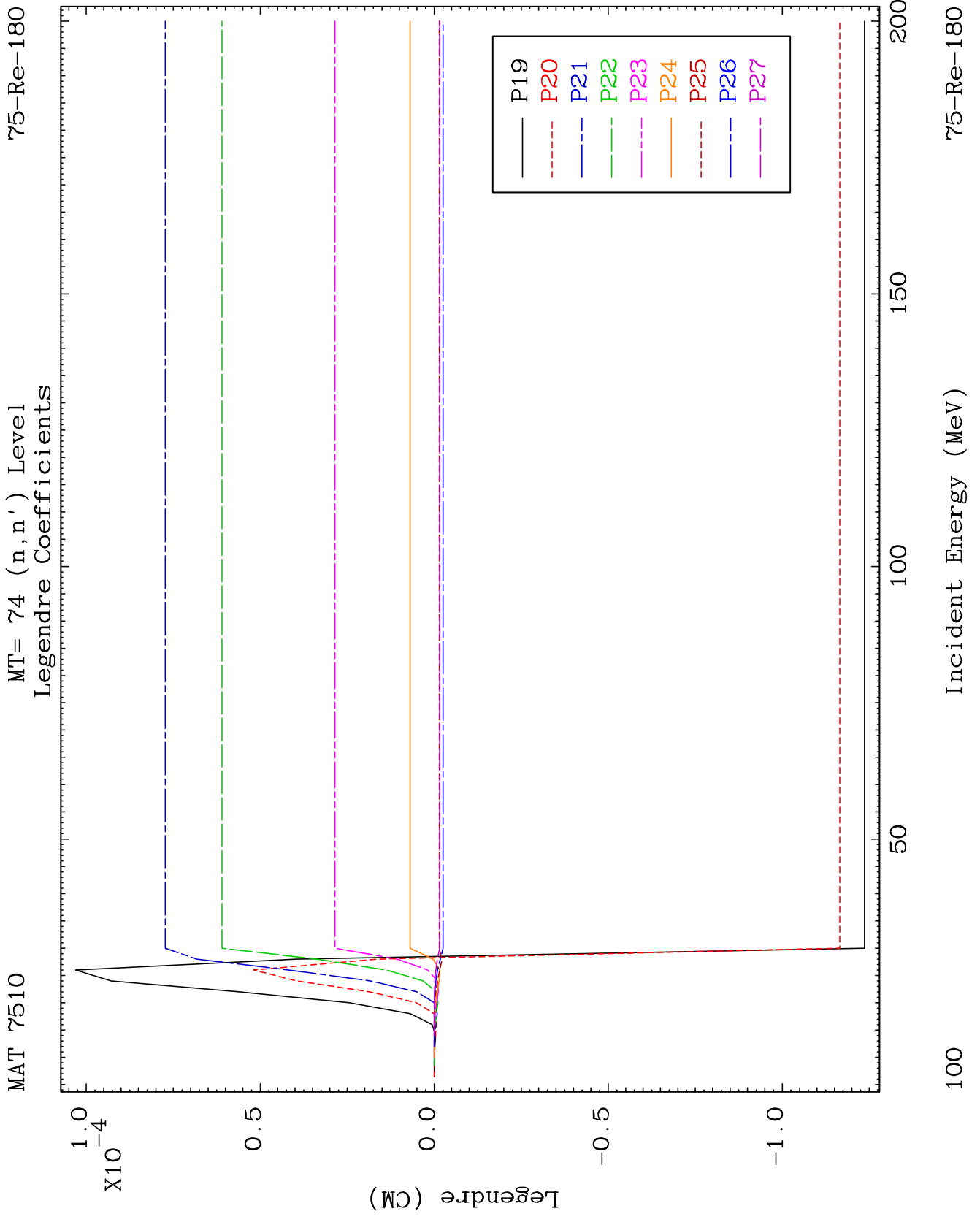
Incident Energy (MeV)

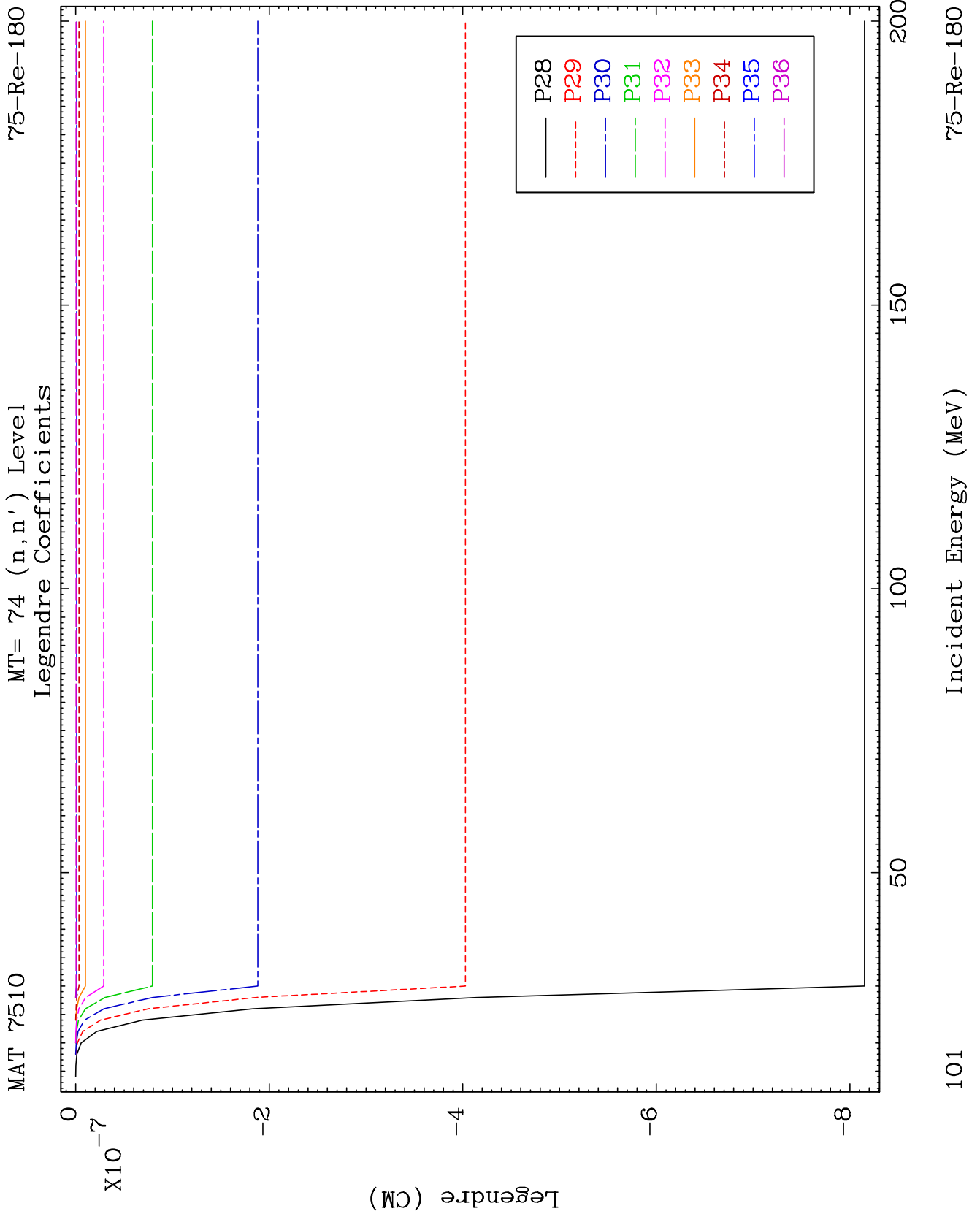
75-Re-180



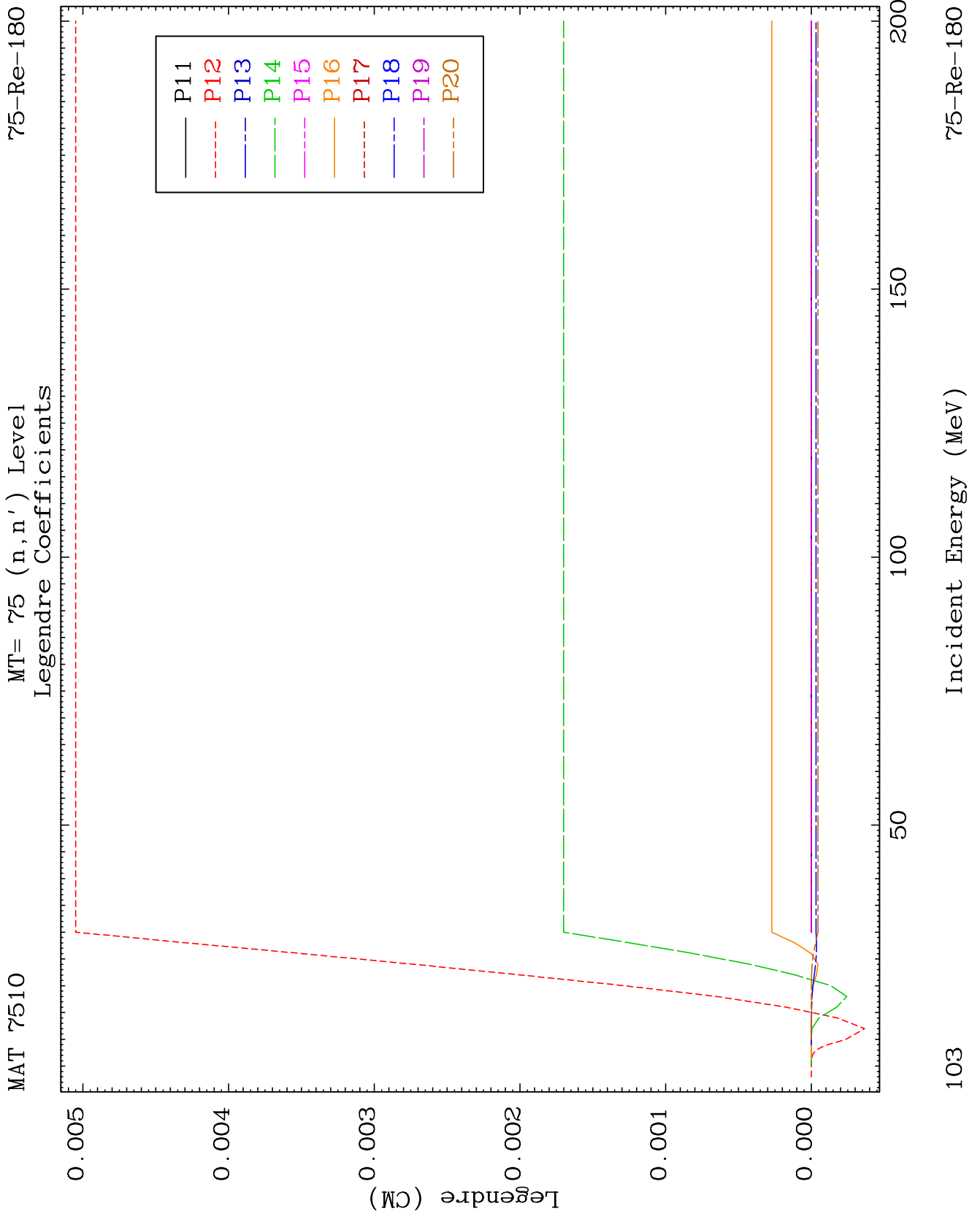




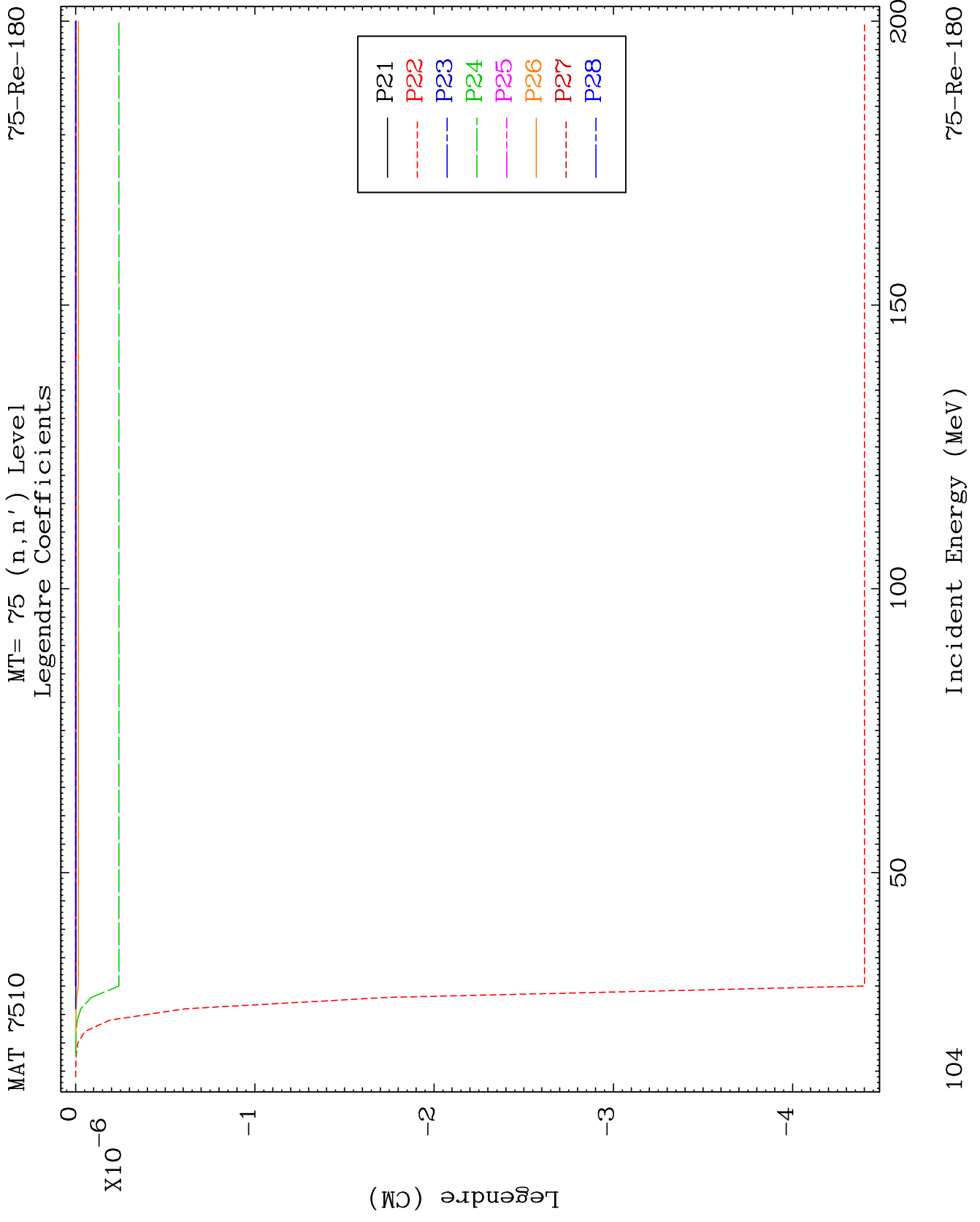


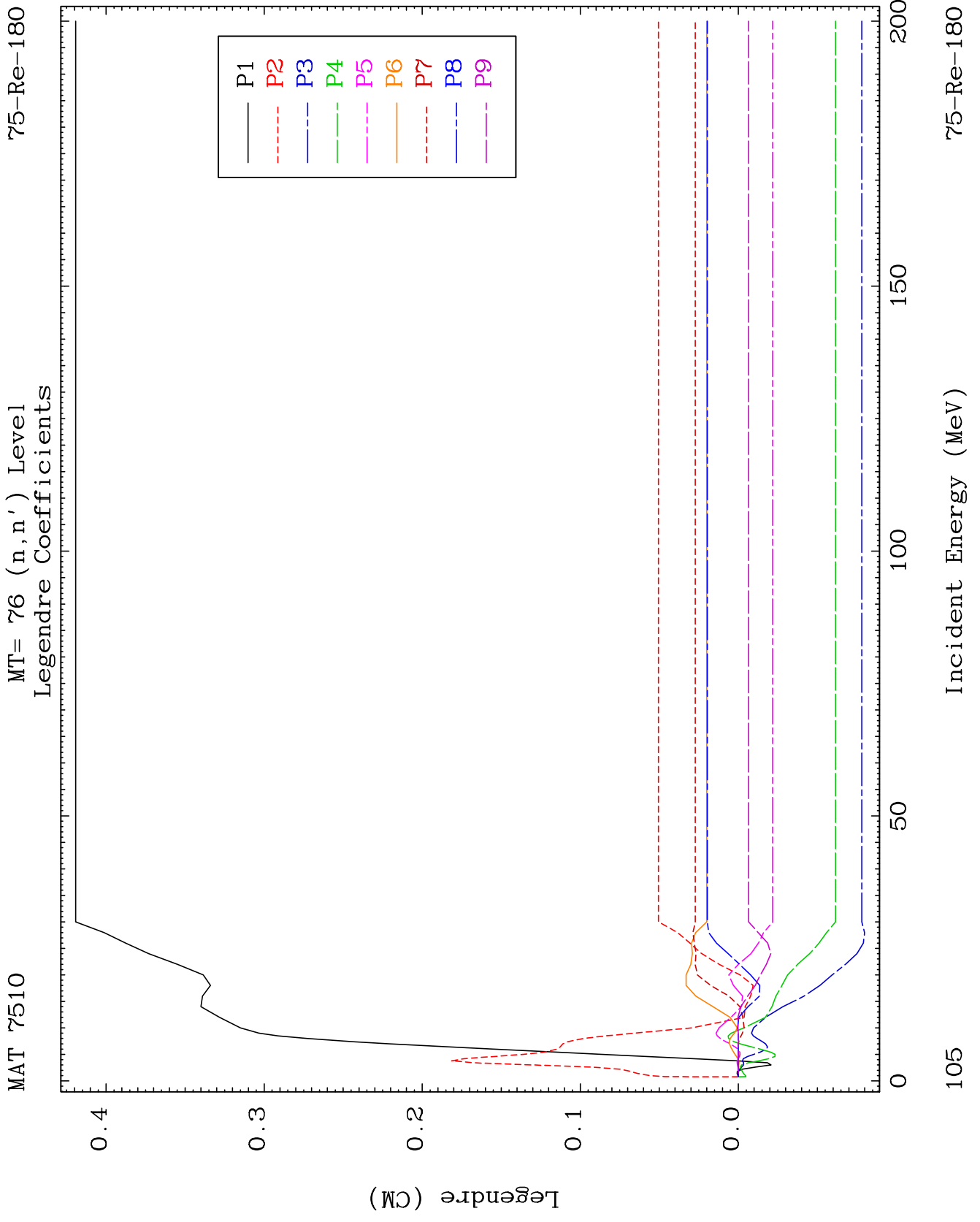


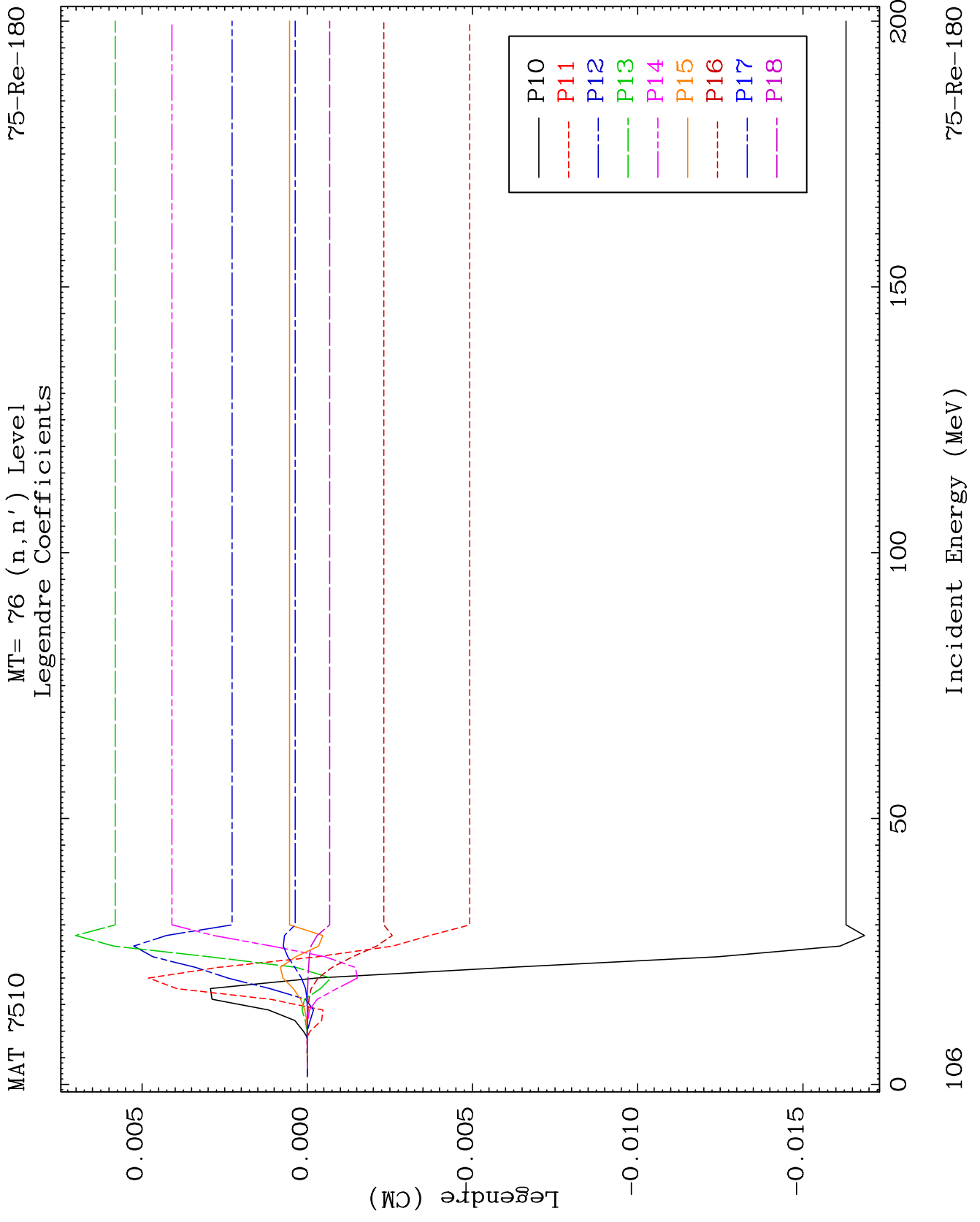


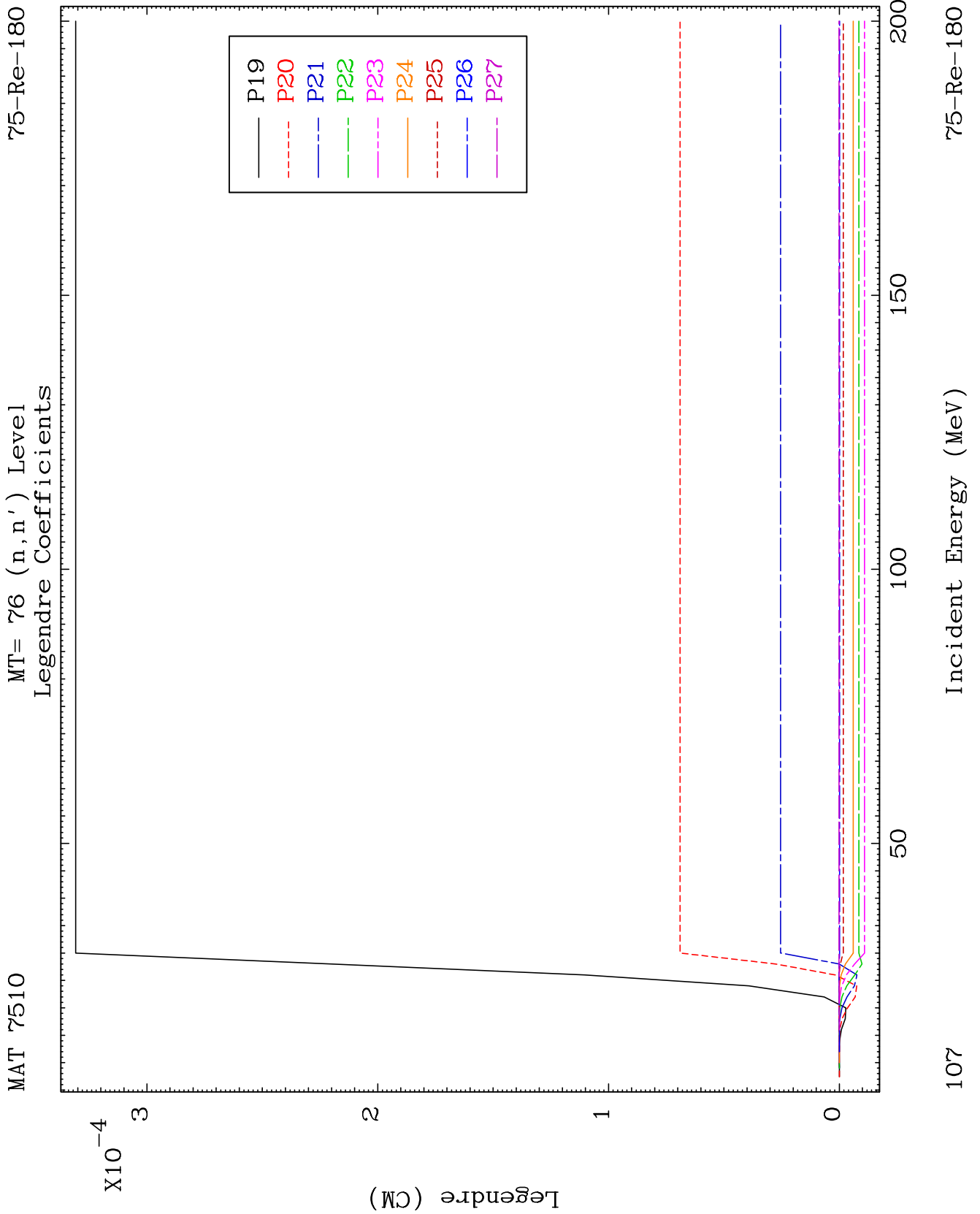


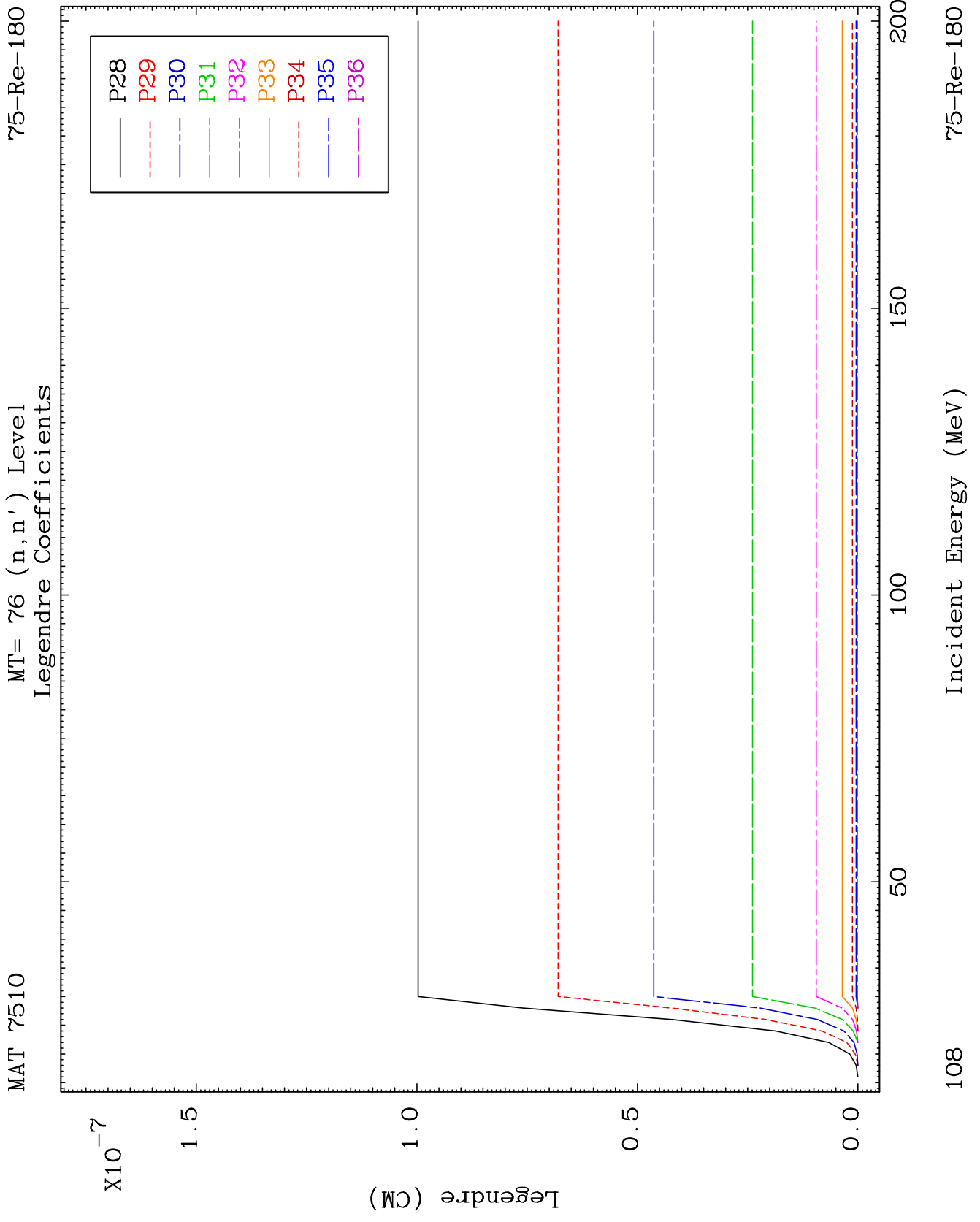


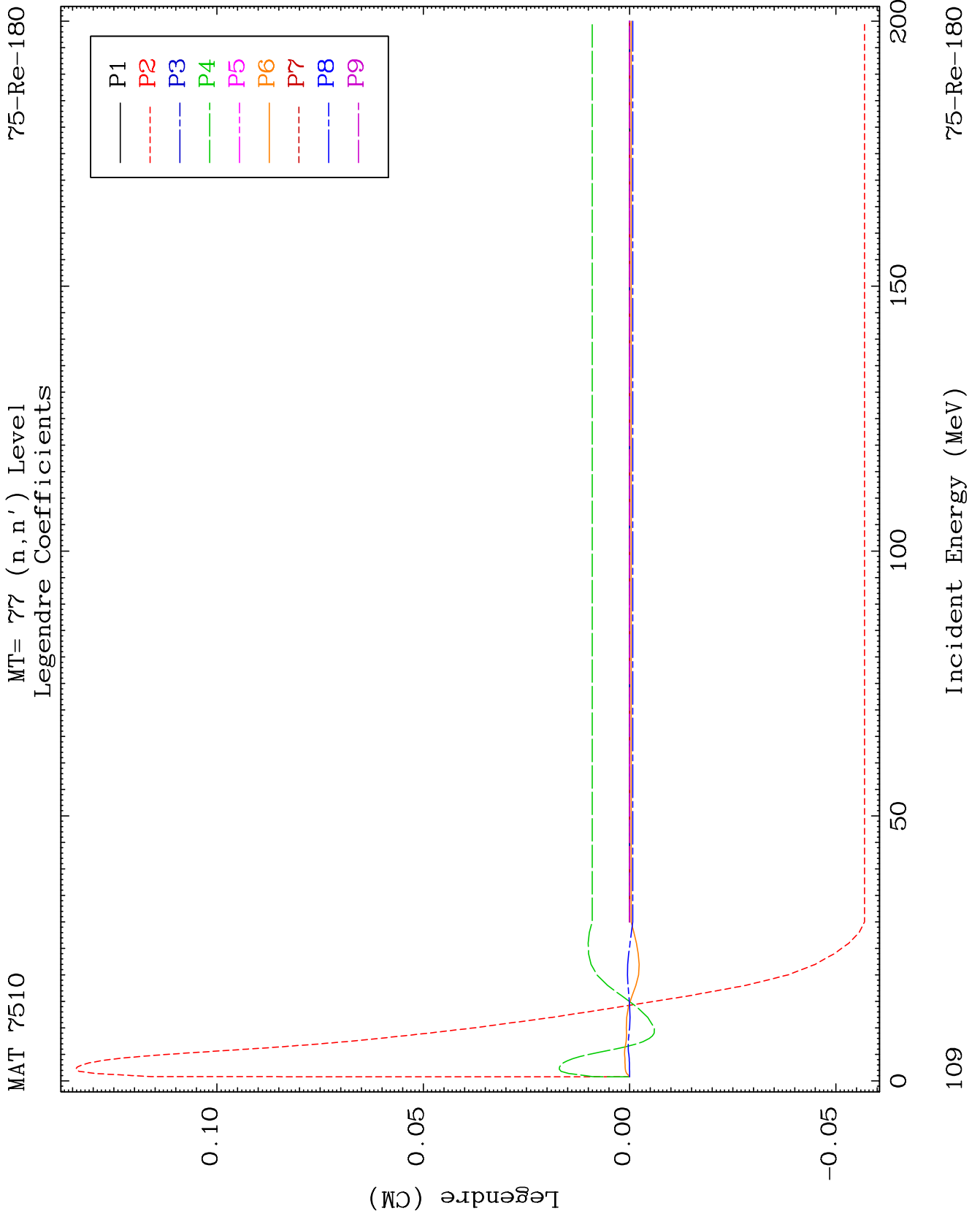


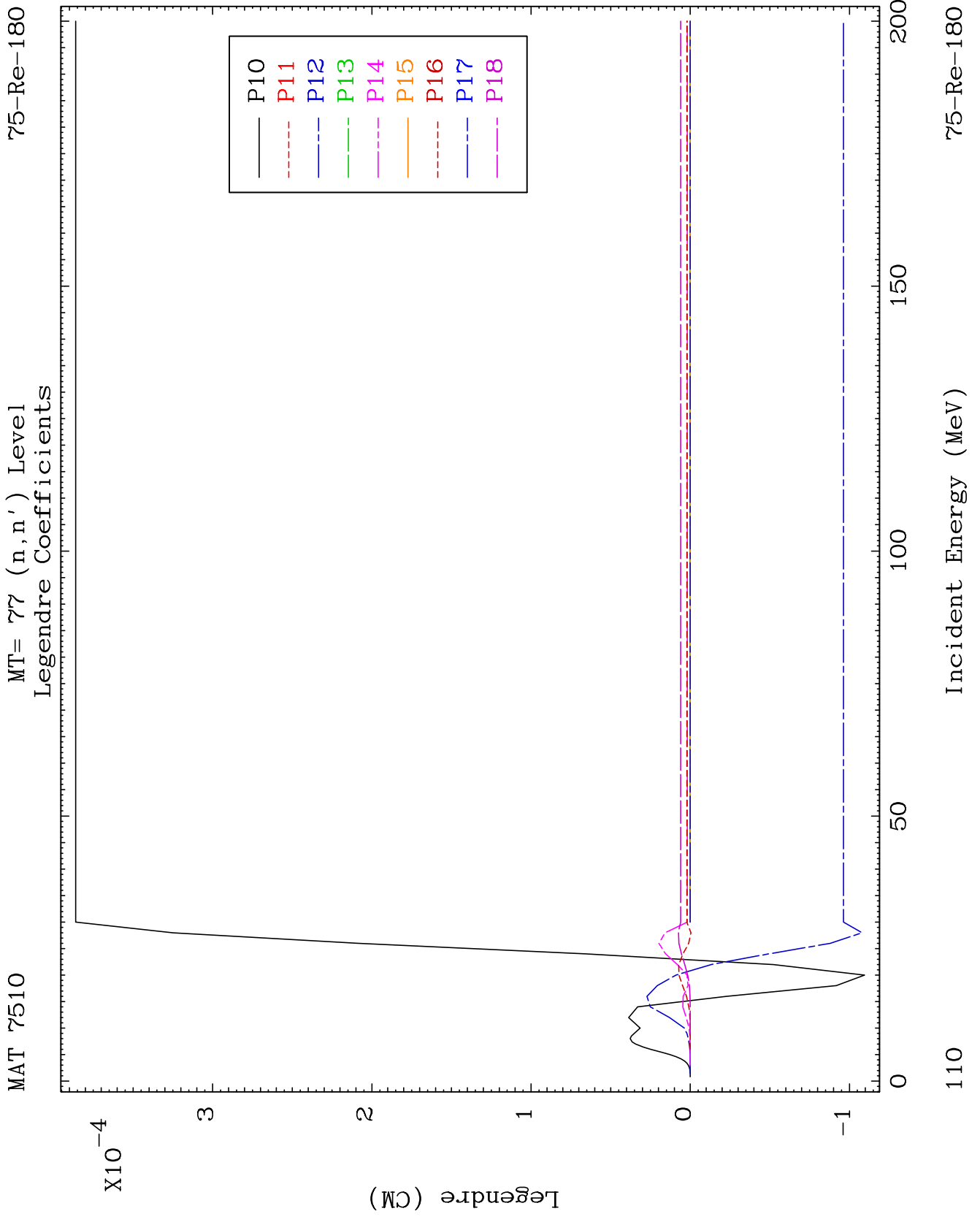


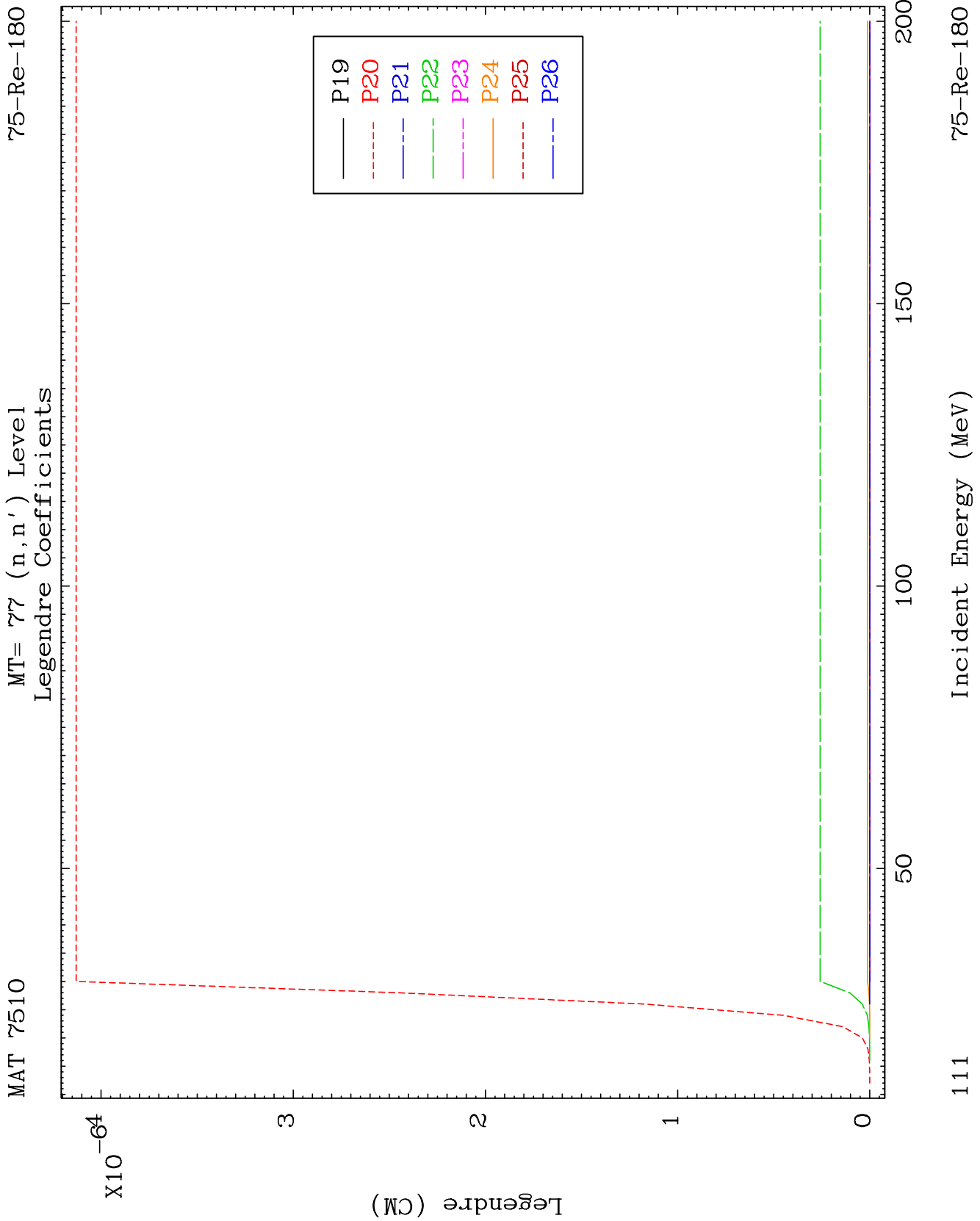




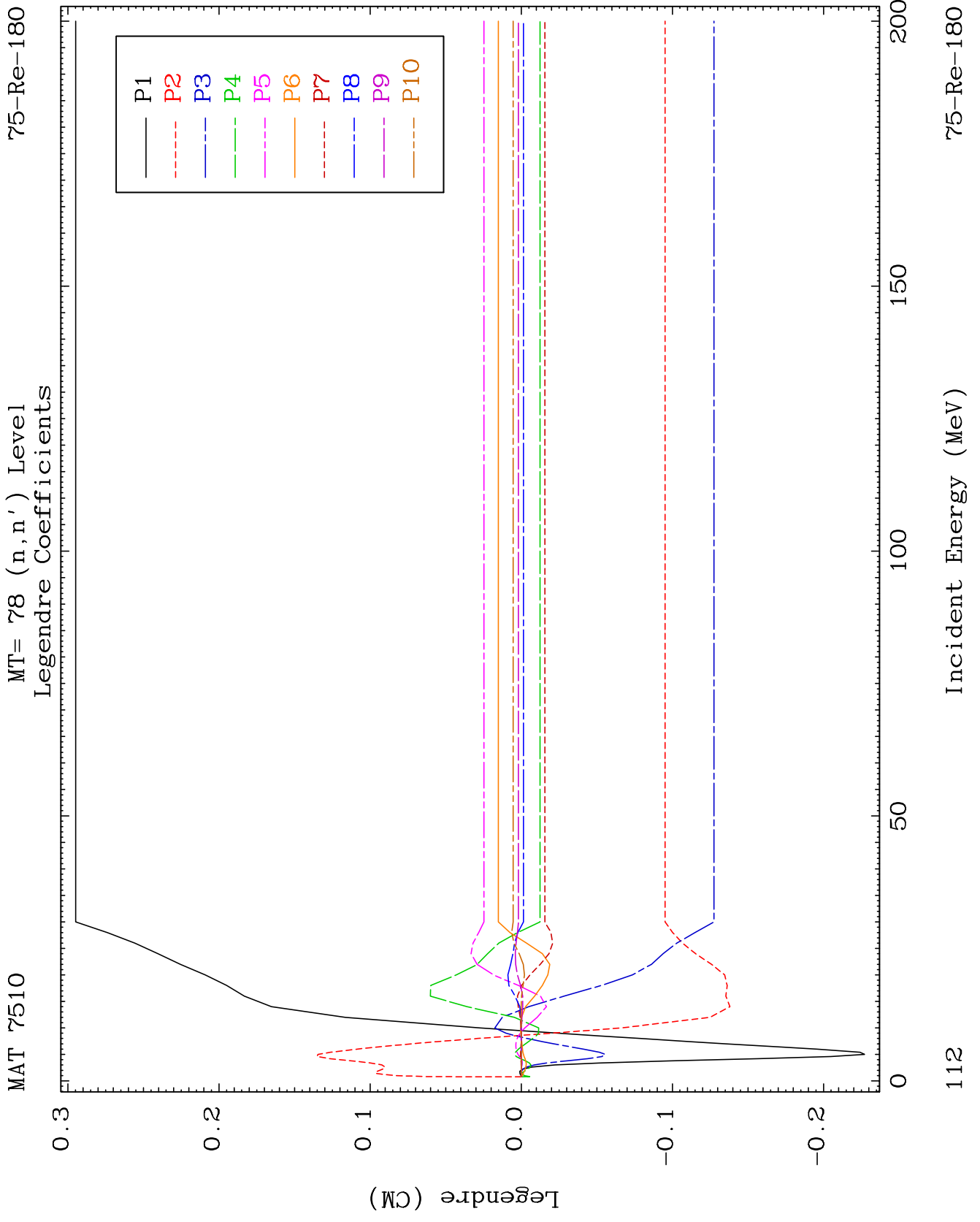


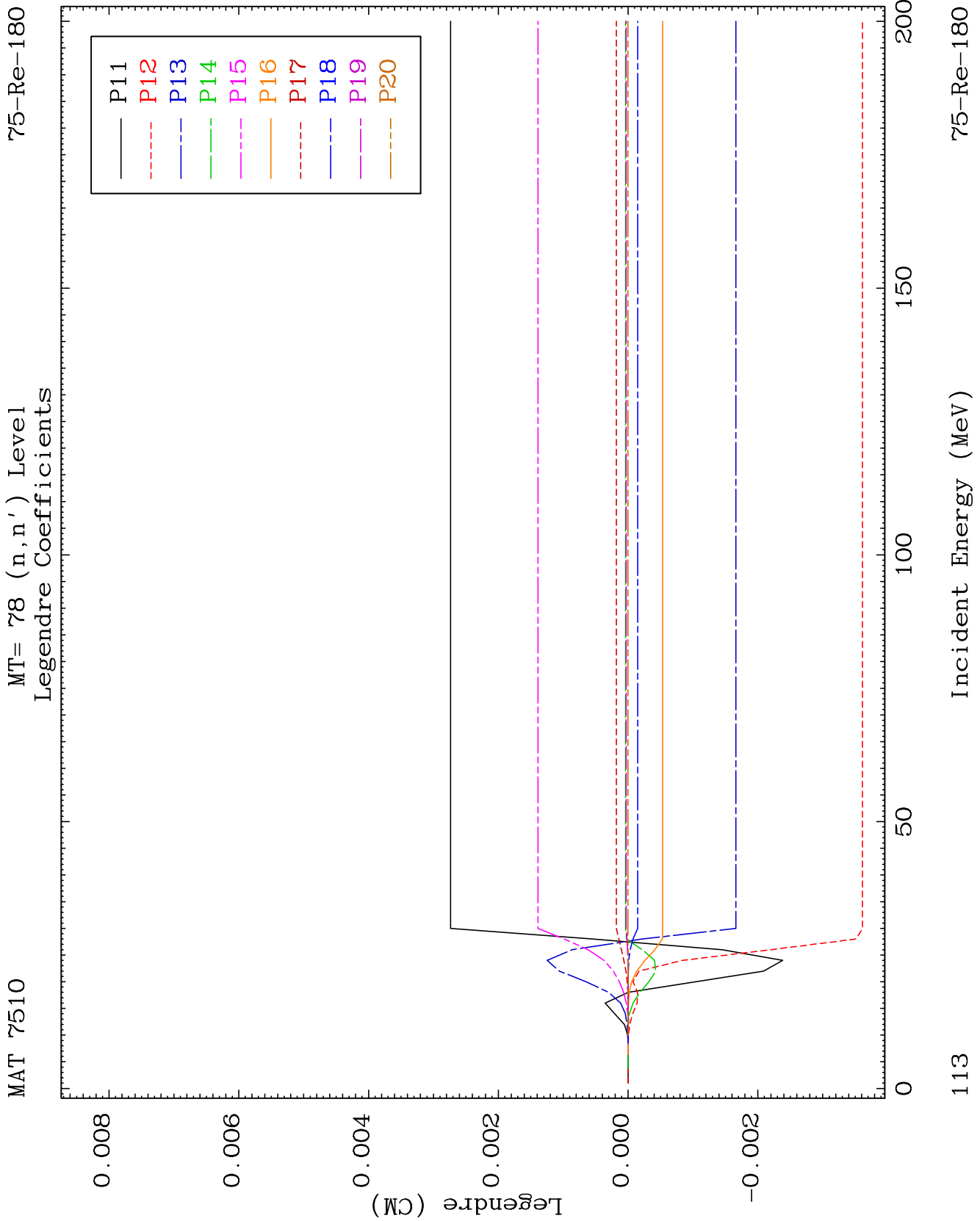


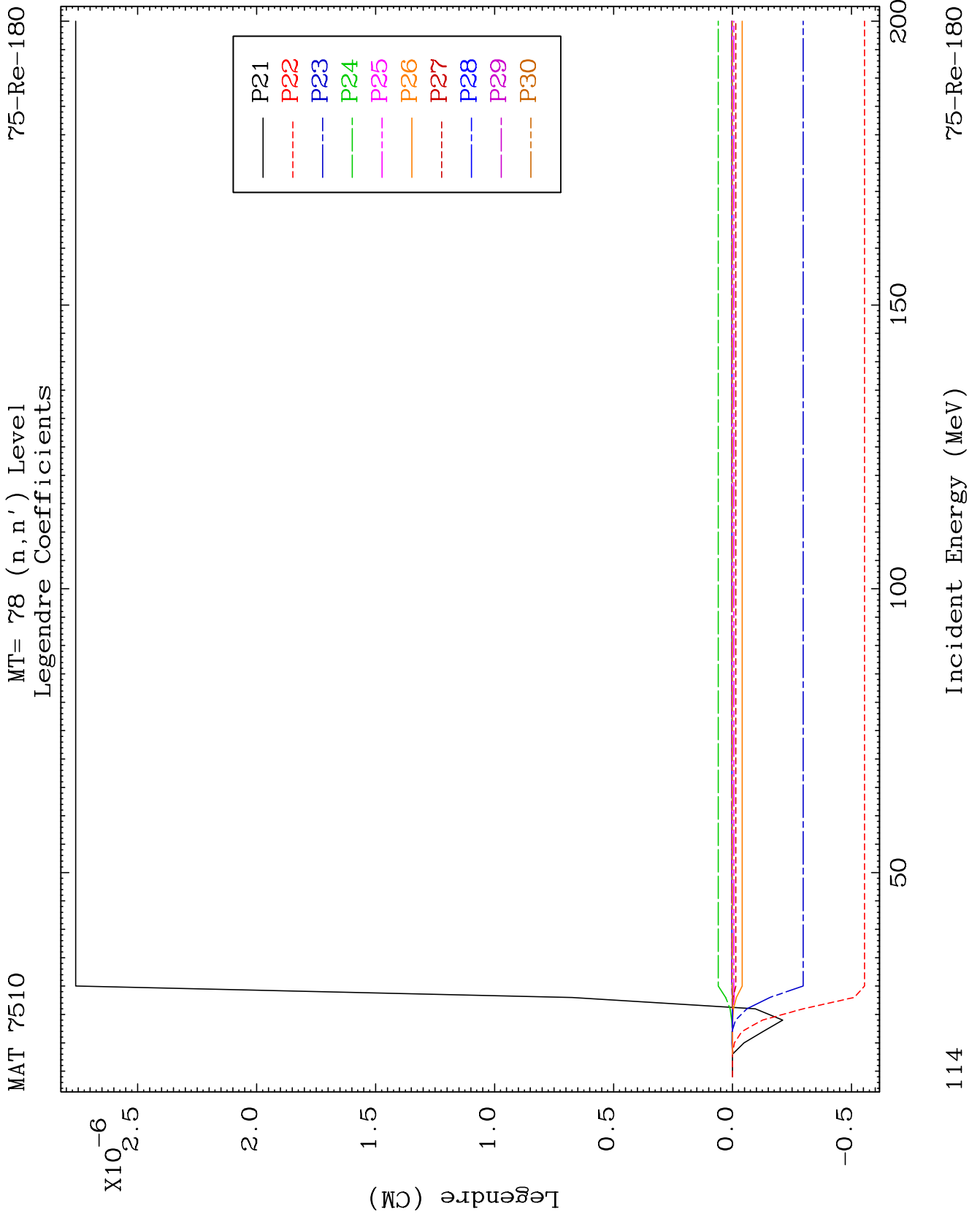


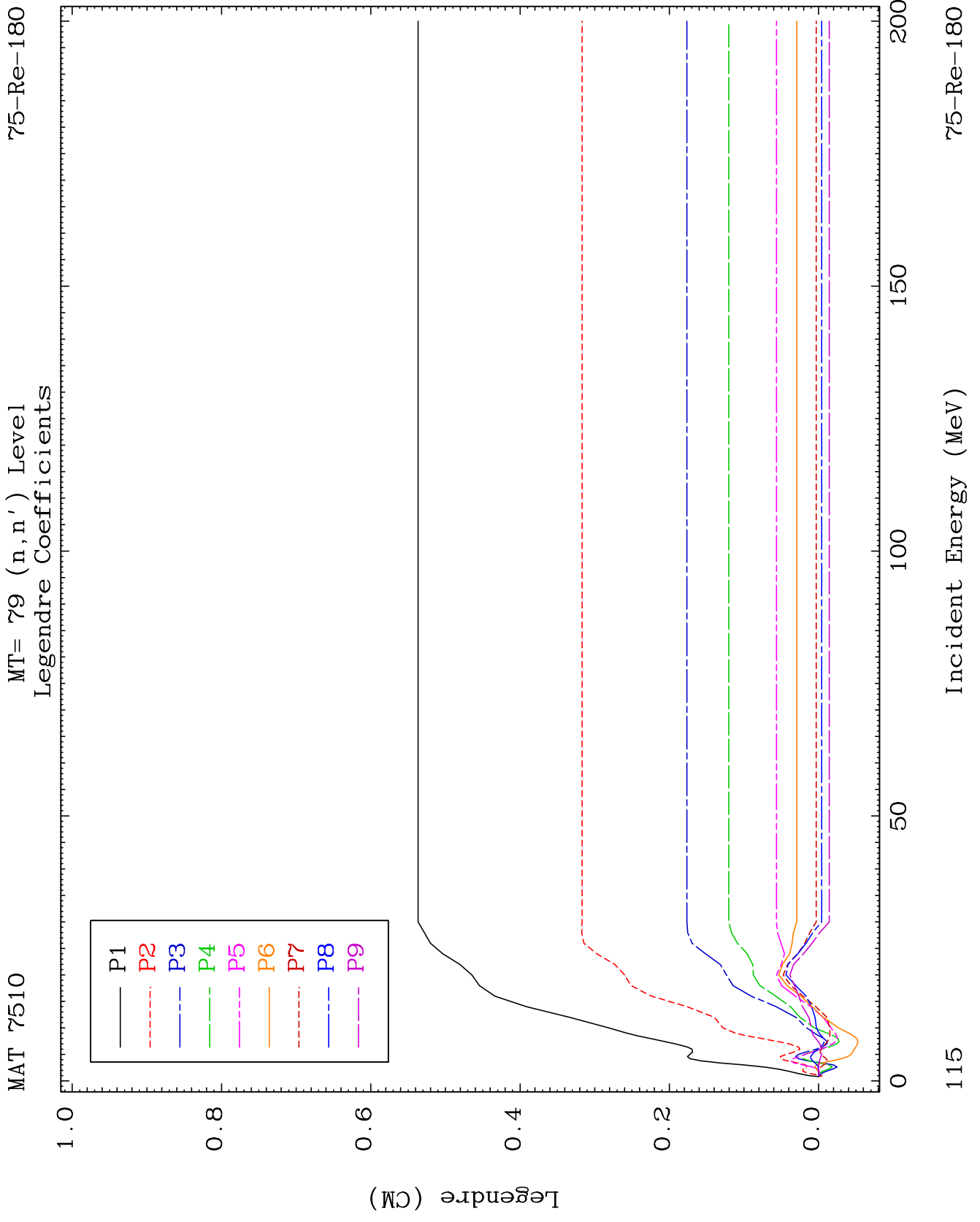


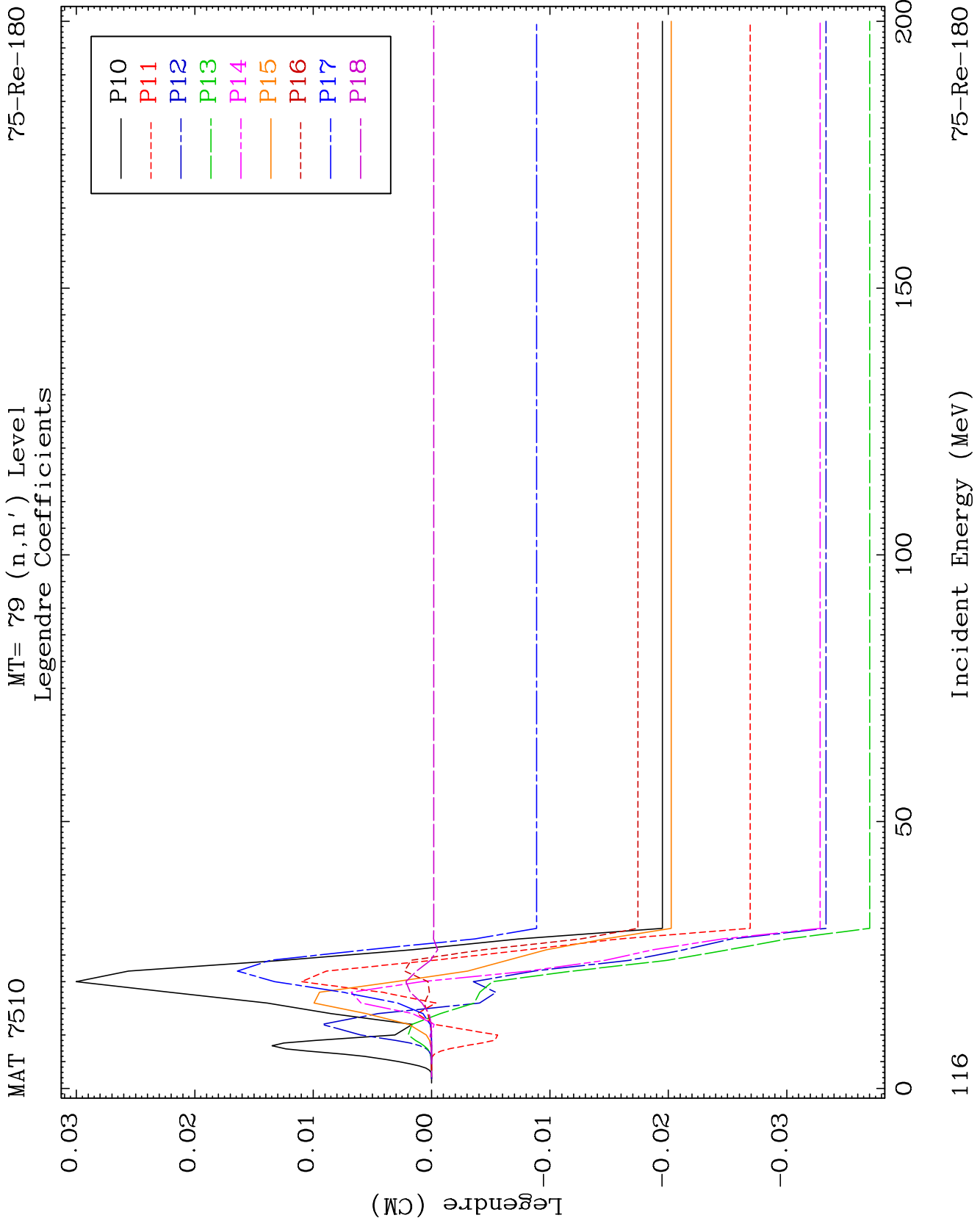


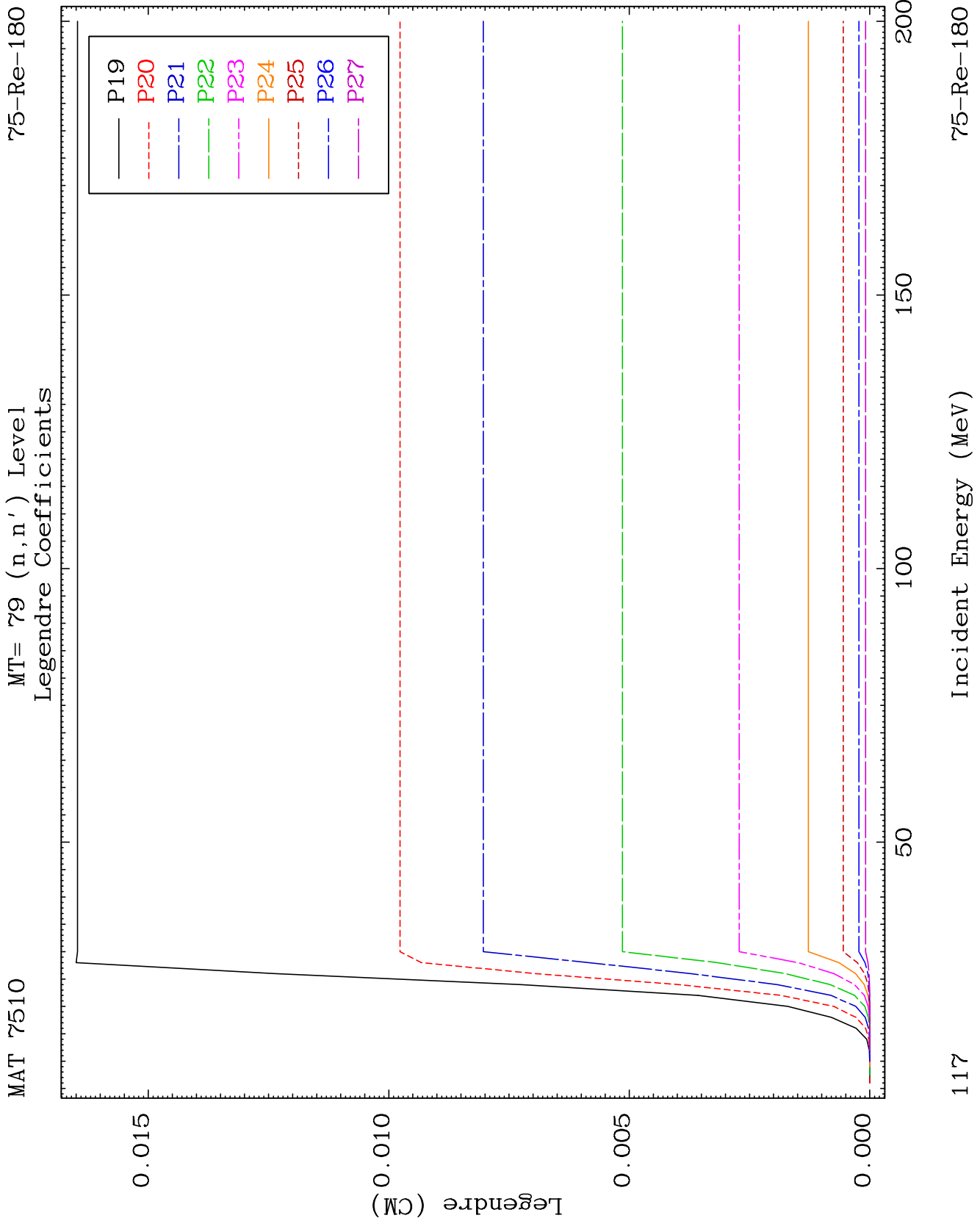


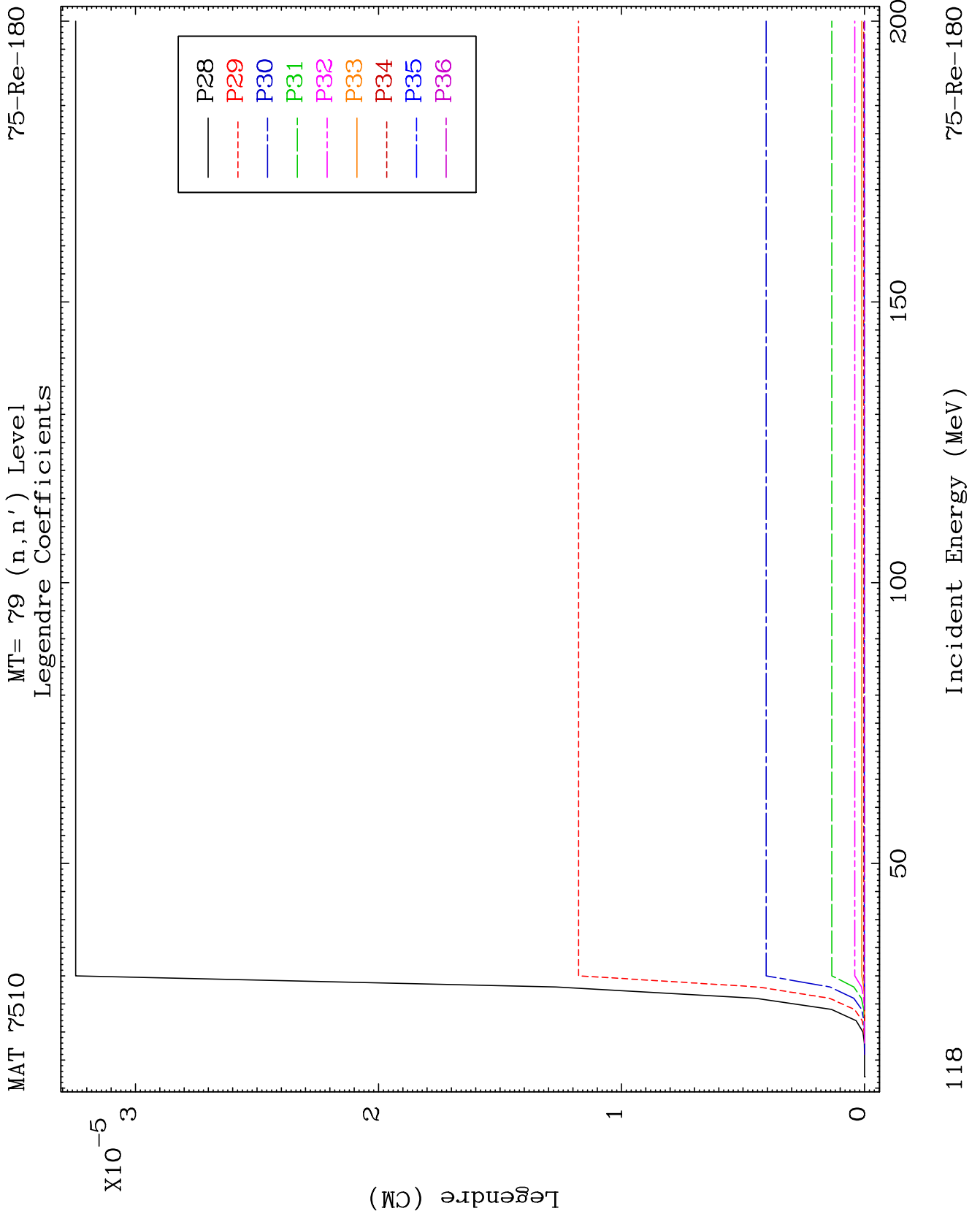










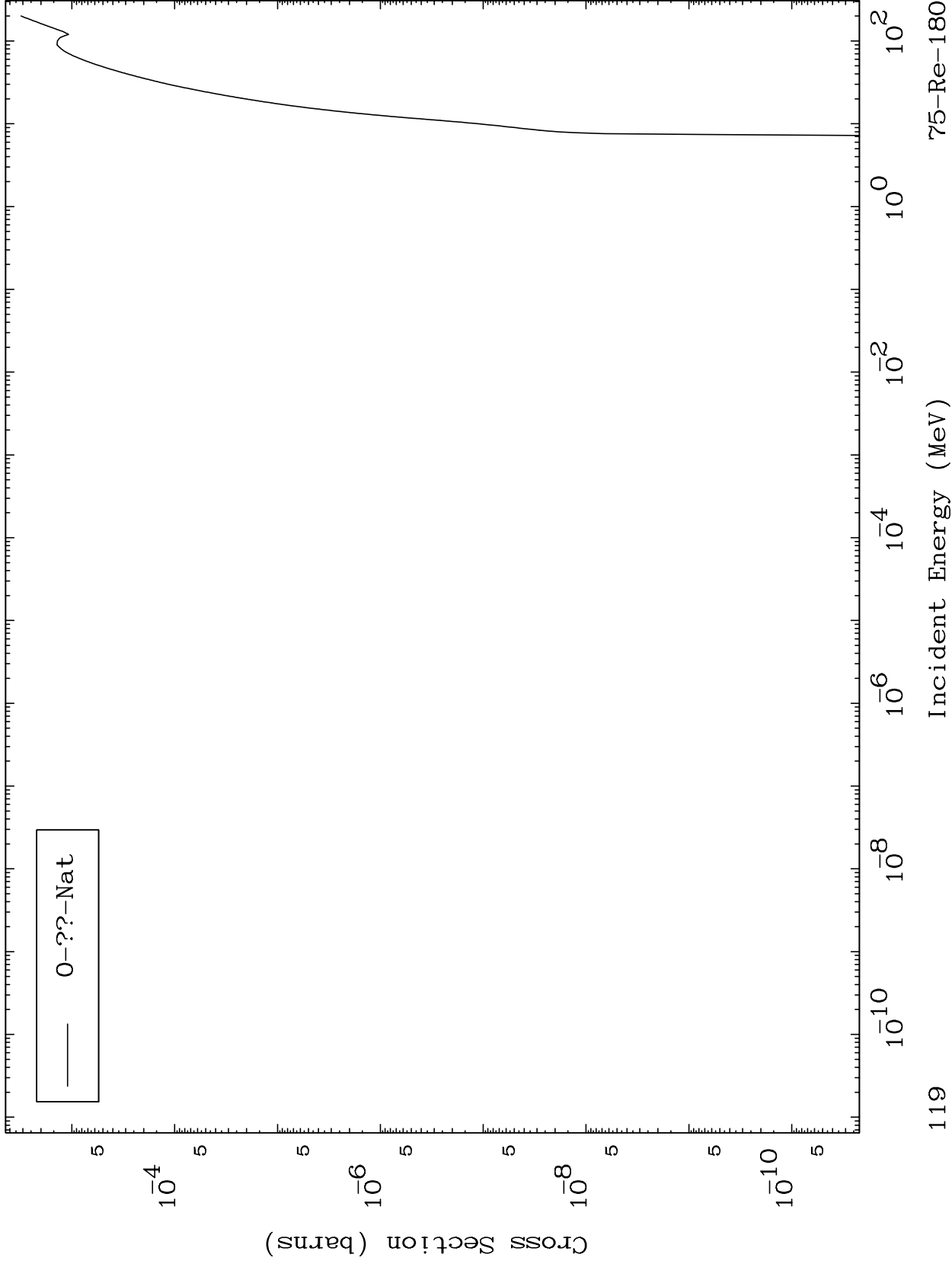


MAT 7510

Fission

<sup>75</sup>Re-180

Radionuclide Production Cross Section



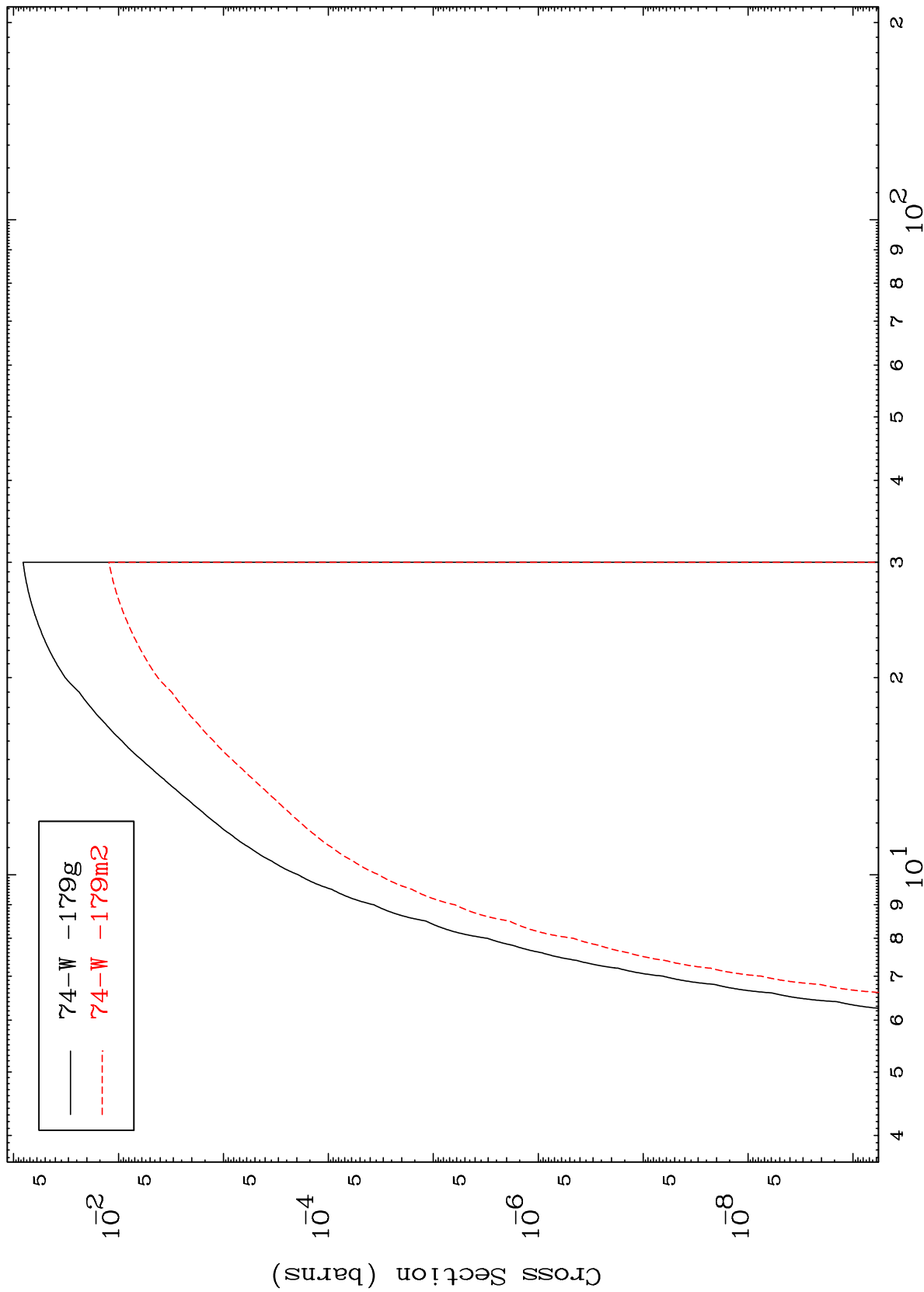


MAT 7510

(n,n') p

75-Re-180

Radionuclide Production Cross Section



74-W -179g  
74-W -179m2

120

Incident Energy (MeV)

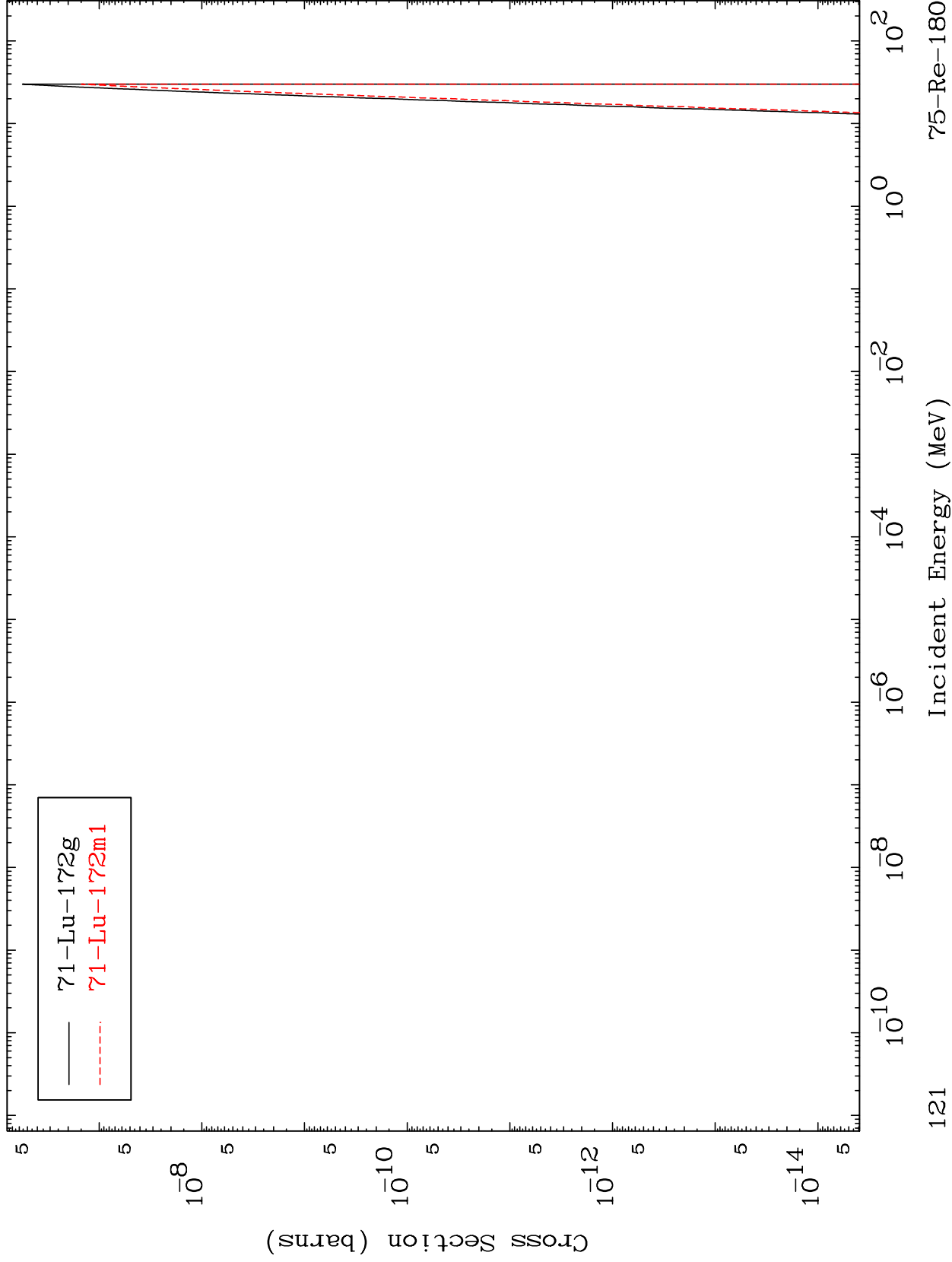
75-Re-180

MAT 7510

$(n, n')$   $2\alpha$

$^{75}\text{Re-180}$

Radionuclide Production Cross Section



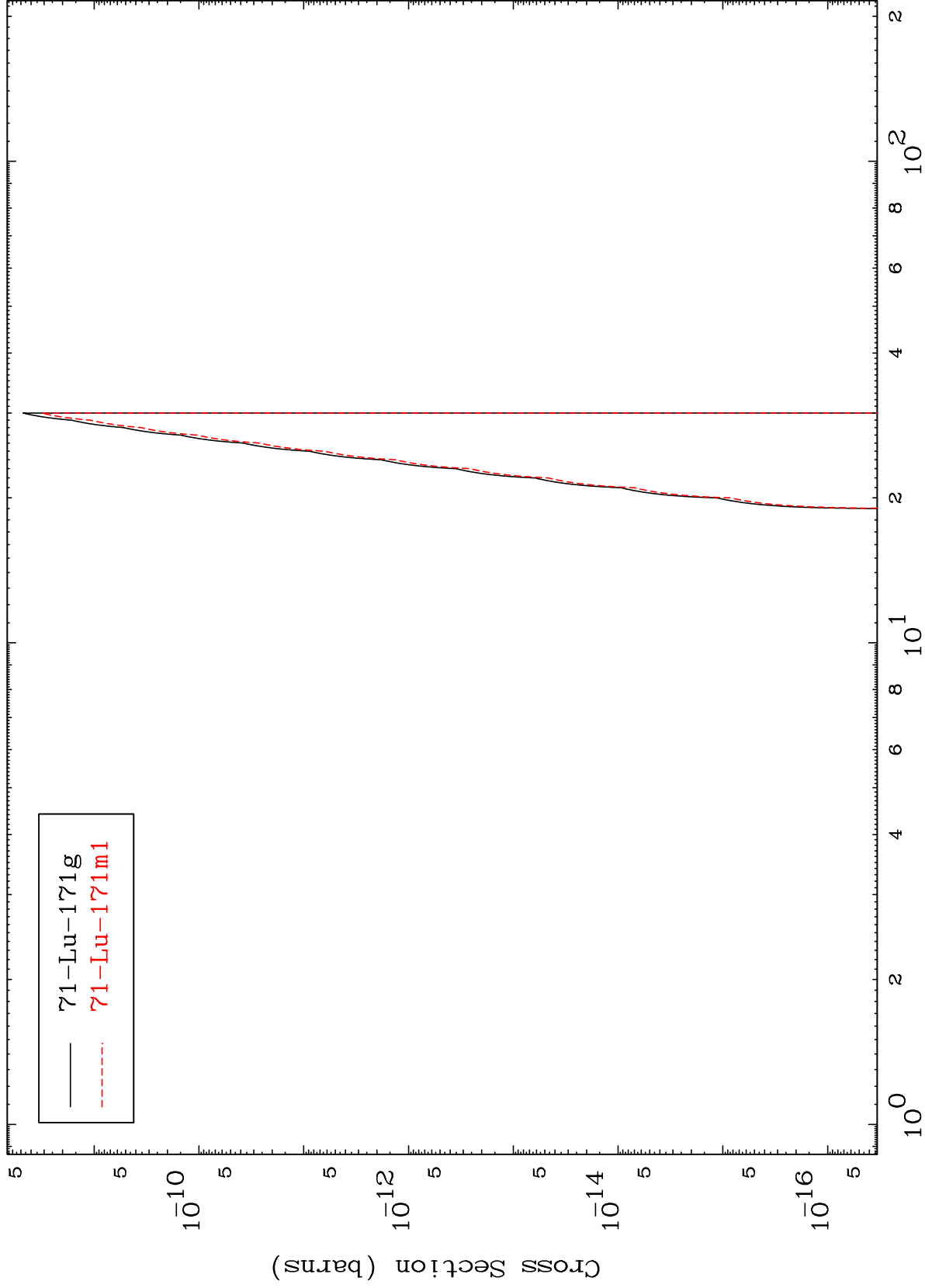
121

MAT 7510

(n,2n) 2α

75-Re-180

Radionuclide Production Cross Section



122

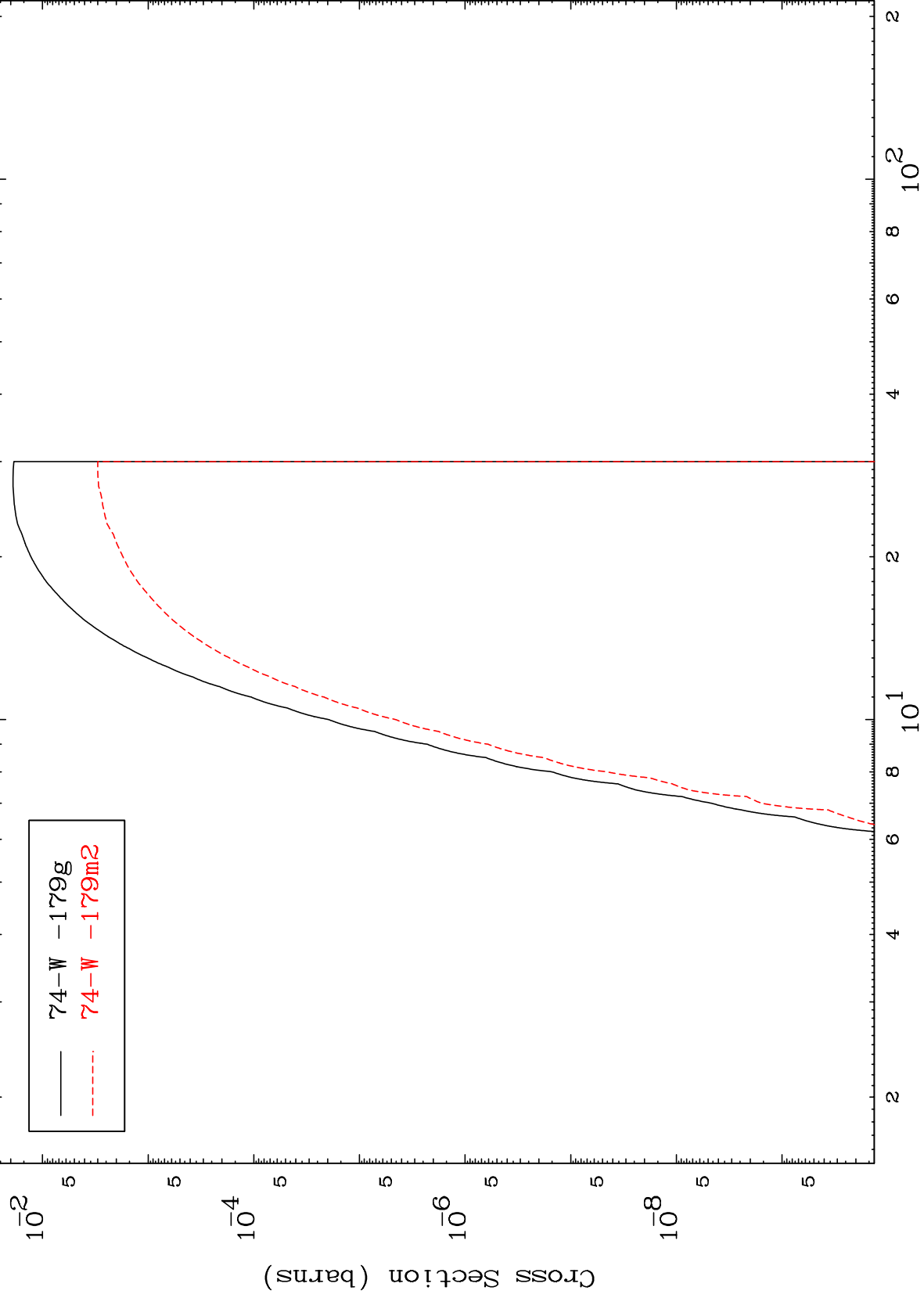
Incident Energy (MeV)

75-Re-180

MAT 7510

75-Re-180

(n,d)  
Radionuclide Production Cross Section



123

75-Re-180

Incident Energy (MeV)