

Program EVALPLOT
(Version 2021-1)

by

Dermott E. Cullen
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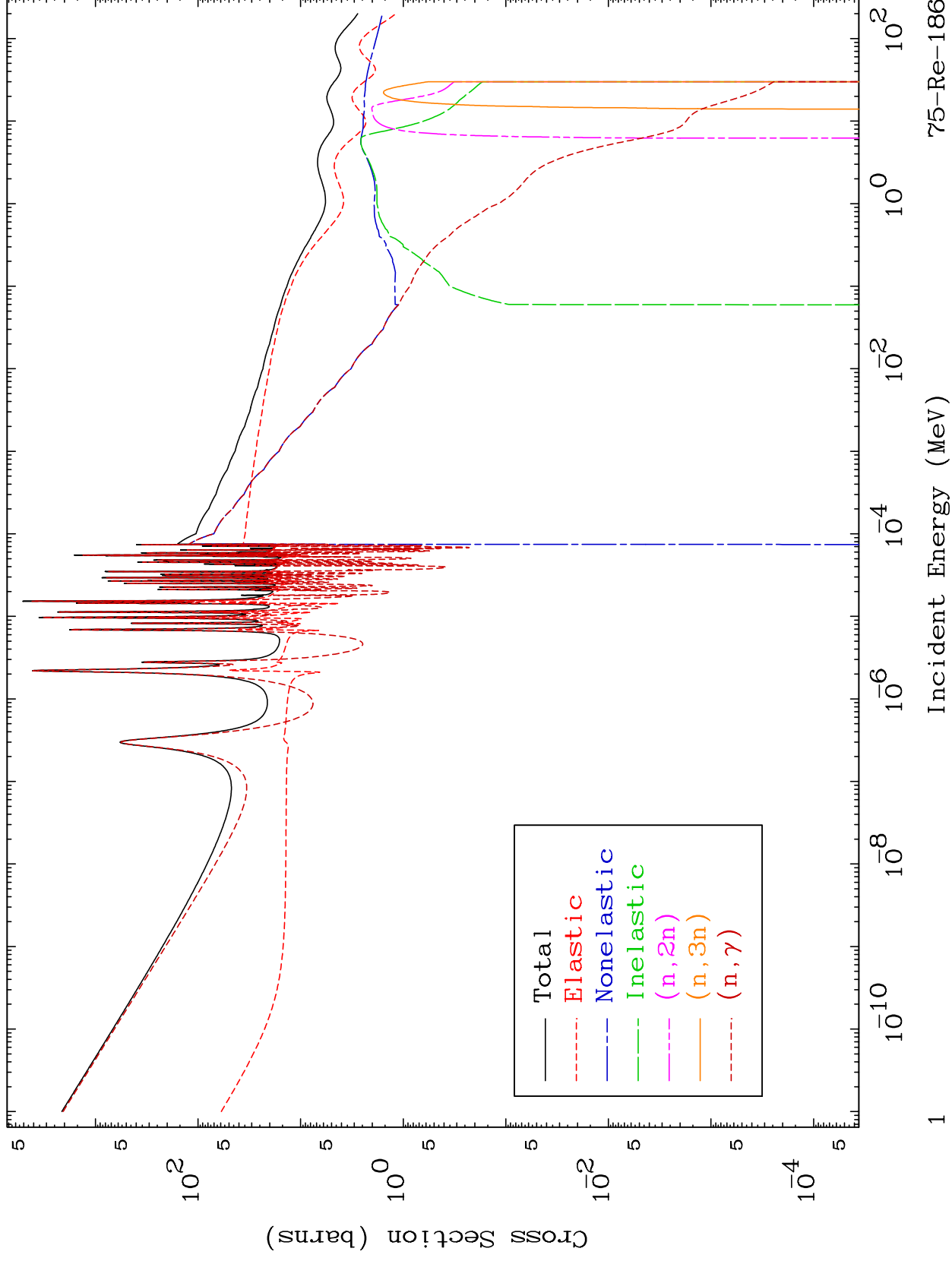
E.Mail: redcullen1@comcast.net
Web: redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 7528

Neutron Major
293 Kelvin Cross Sections

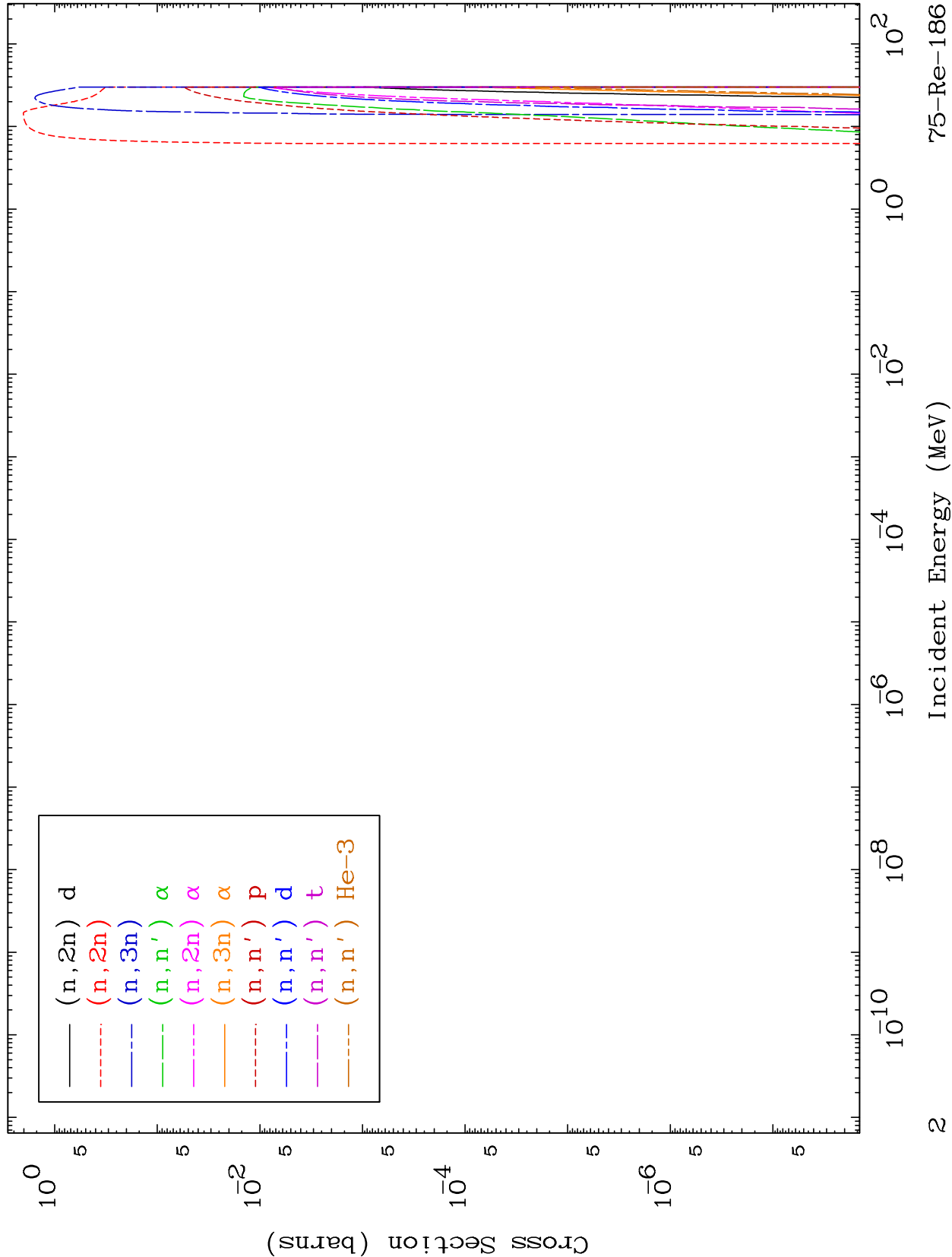
75-Re-186

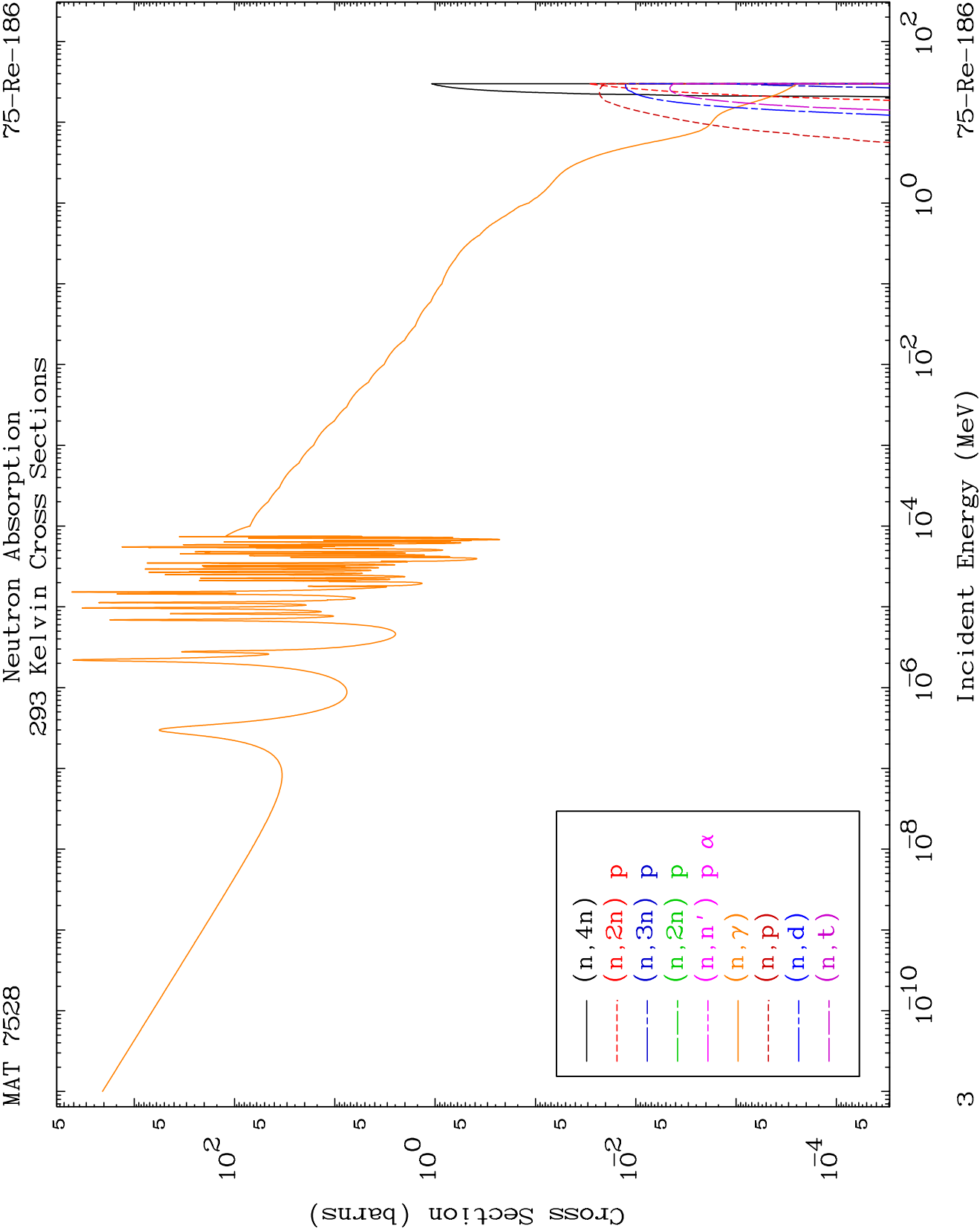


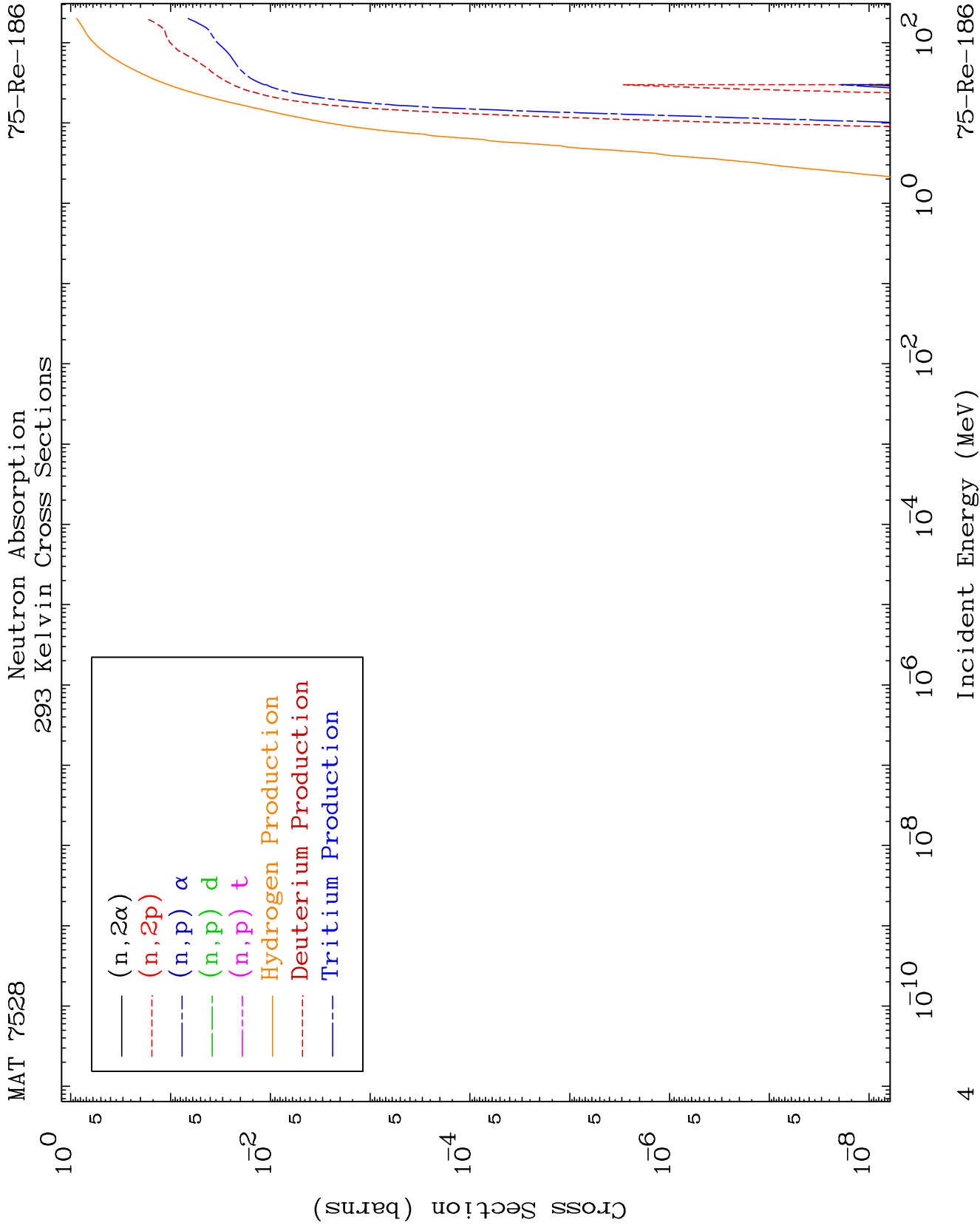
MAT 7528

Neutron Absorption
293 Kelvin Cross Sections

75-Re-186



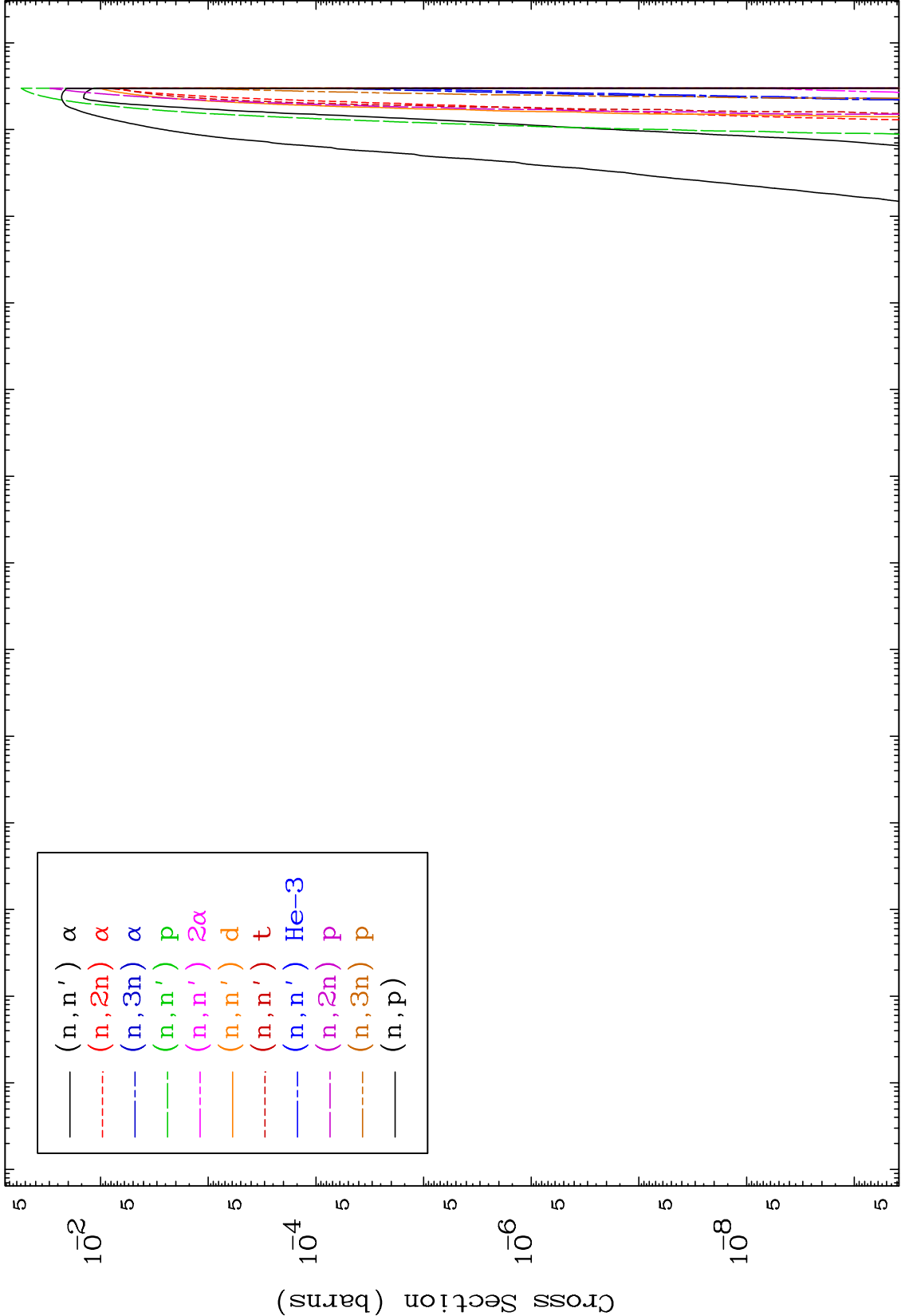


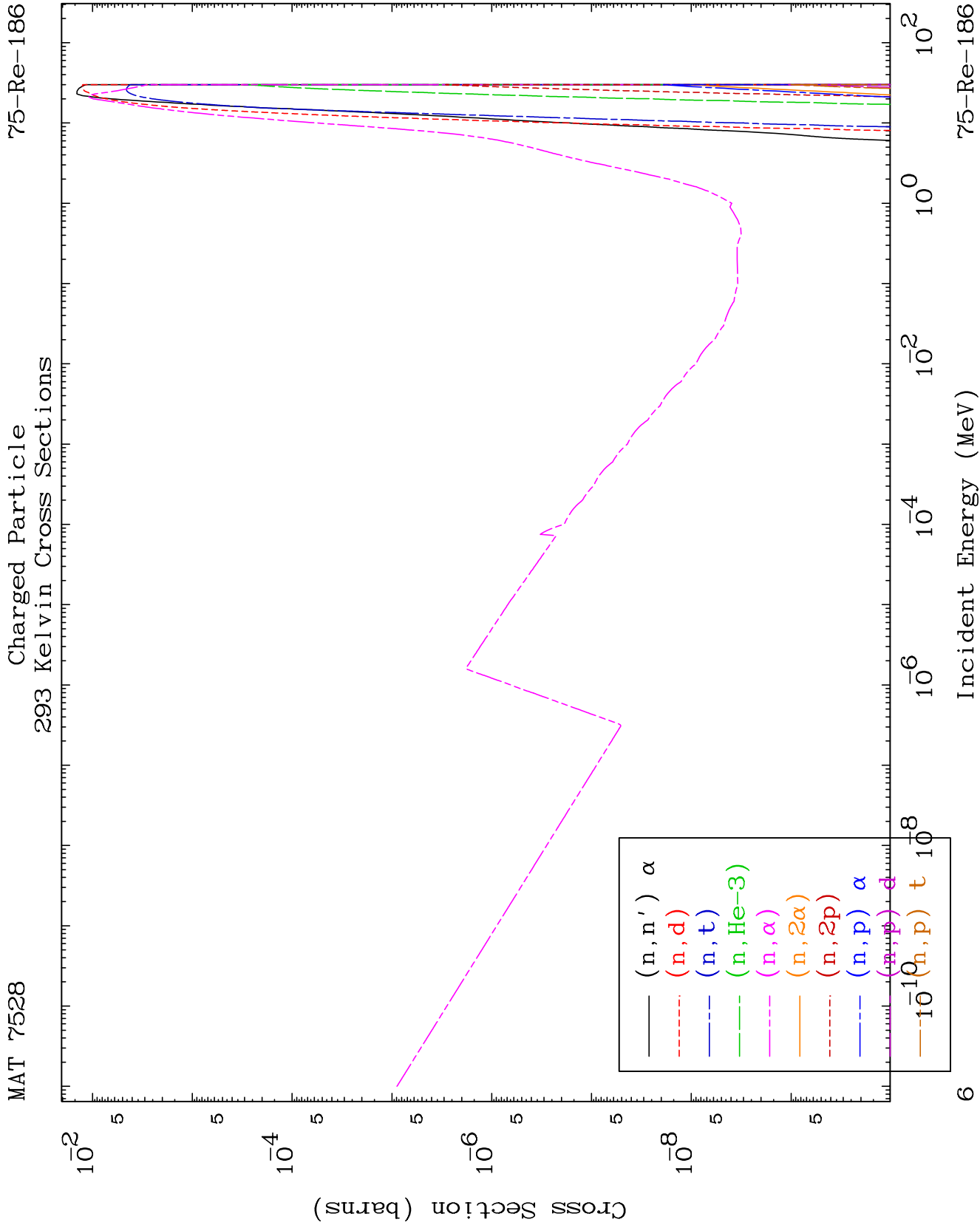


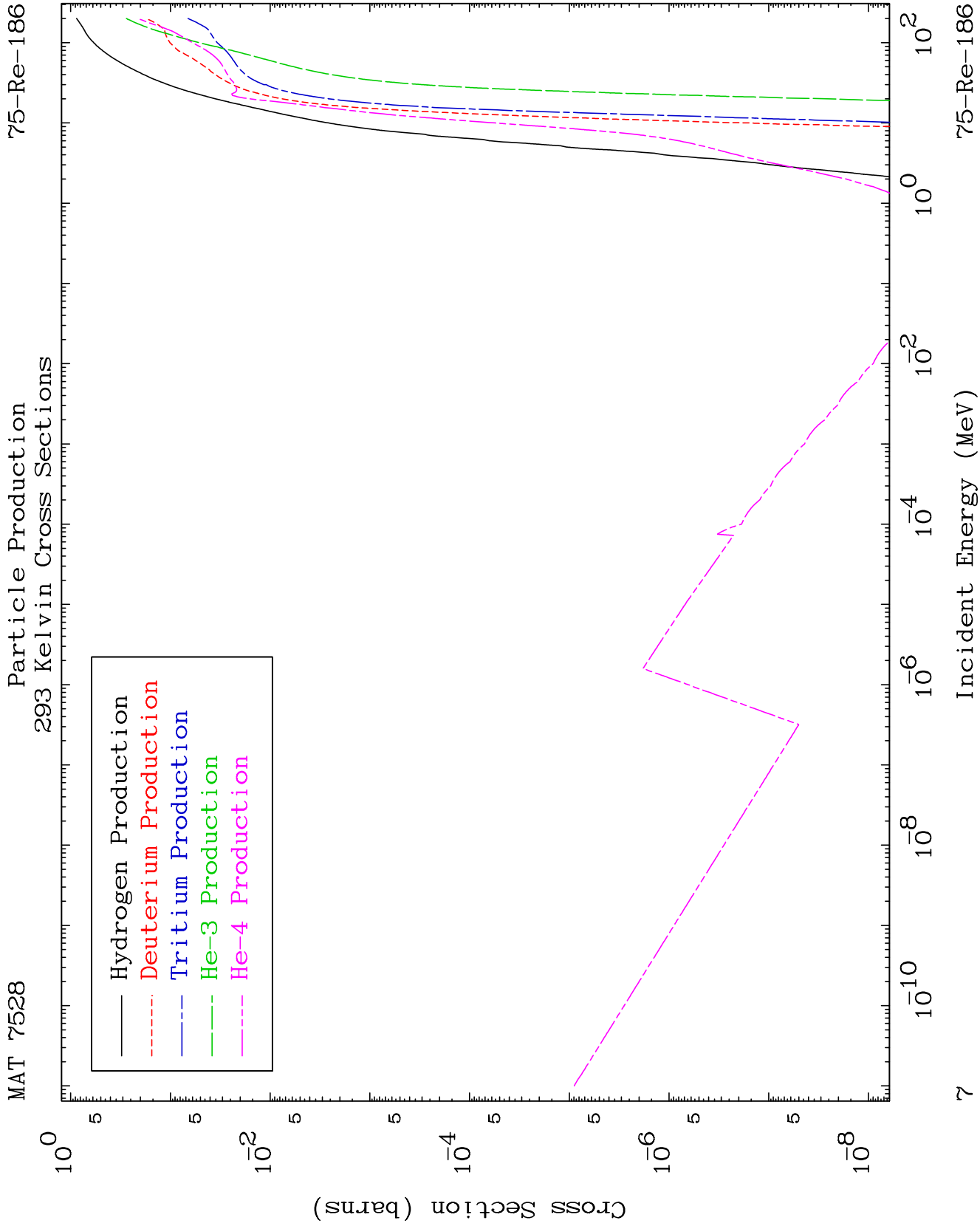
MAT 7528

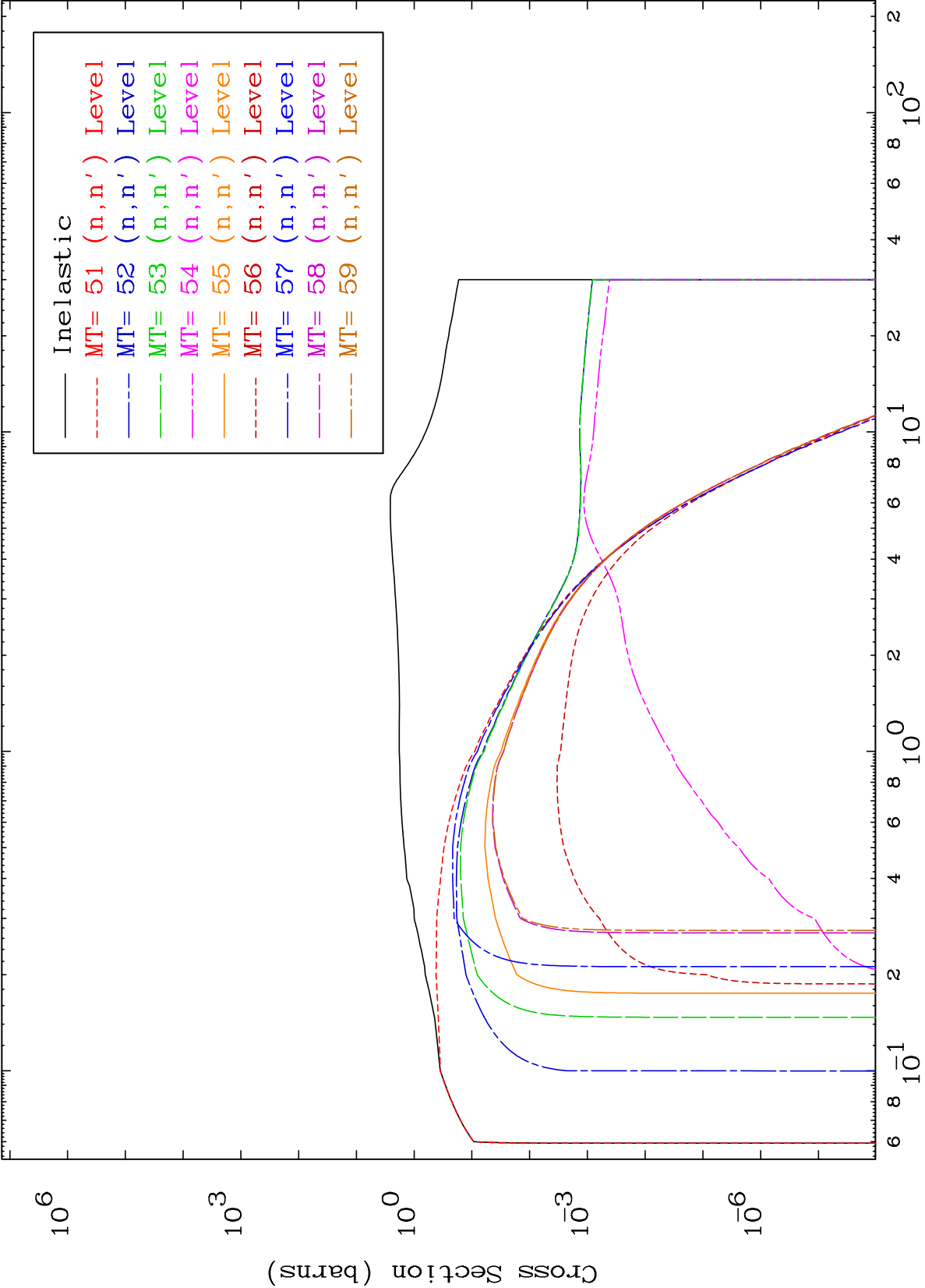
Charged Particle
293 Kelvin Cross Sections

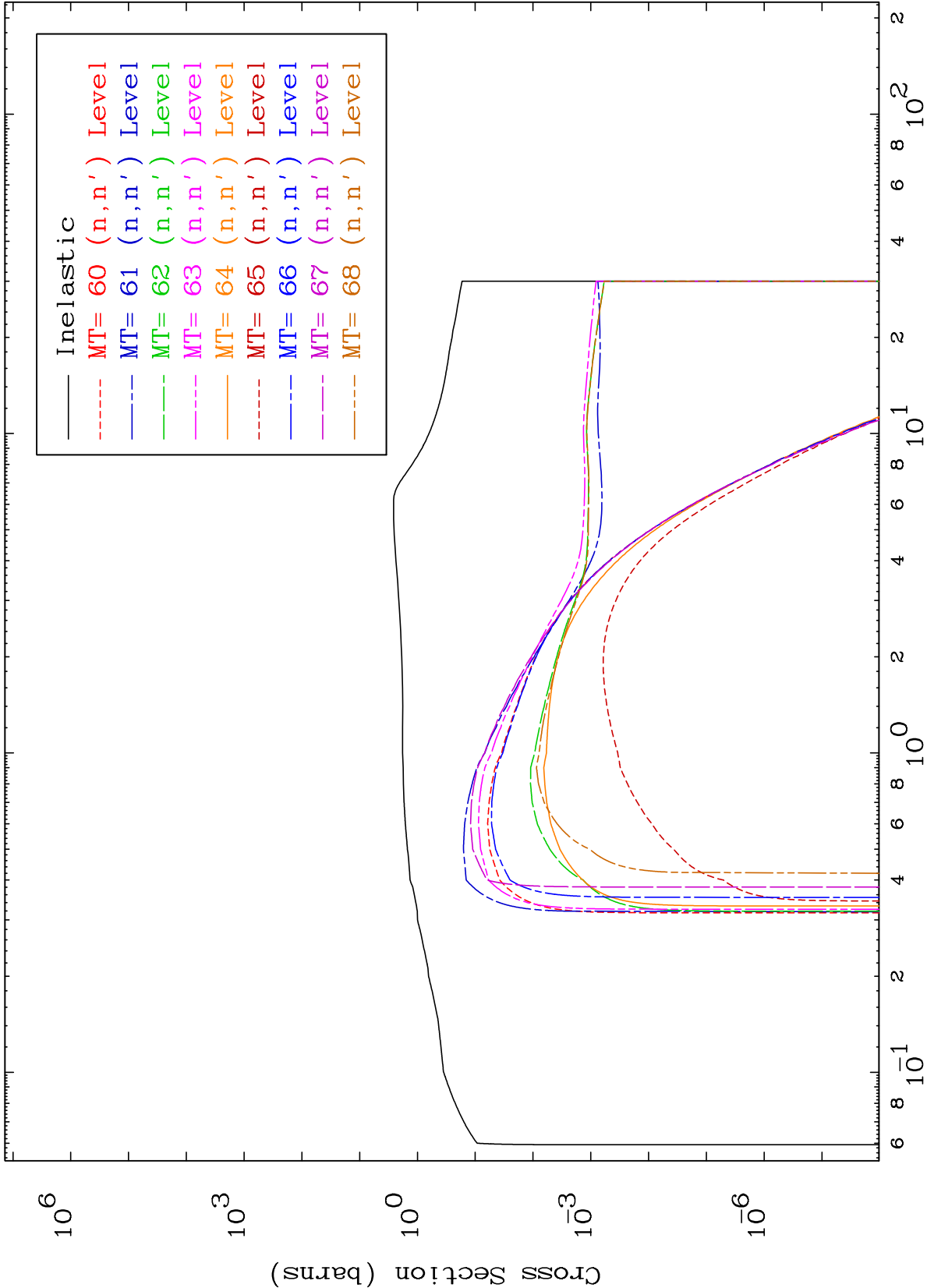
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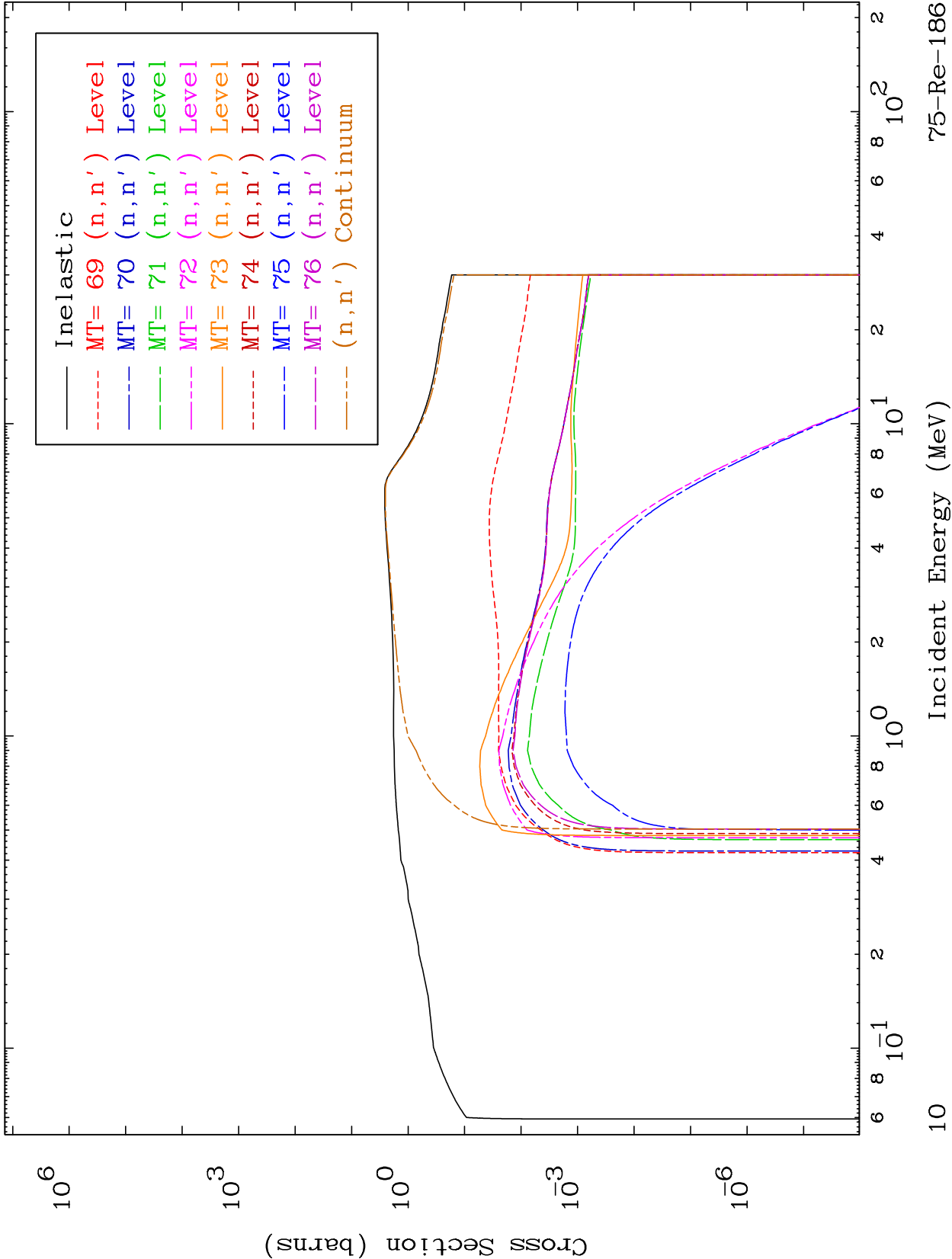


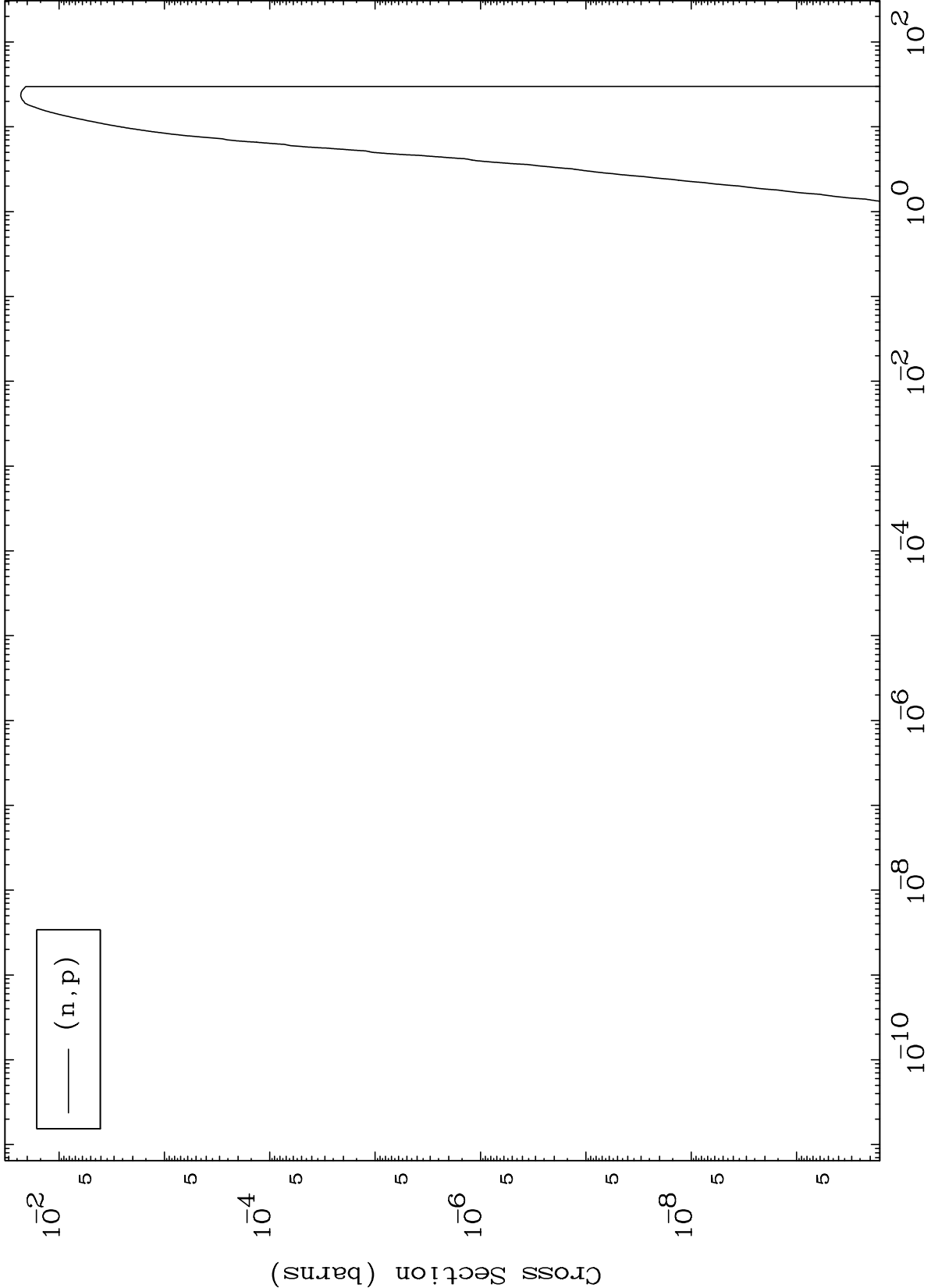


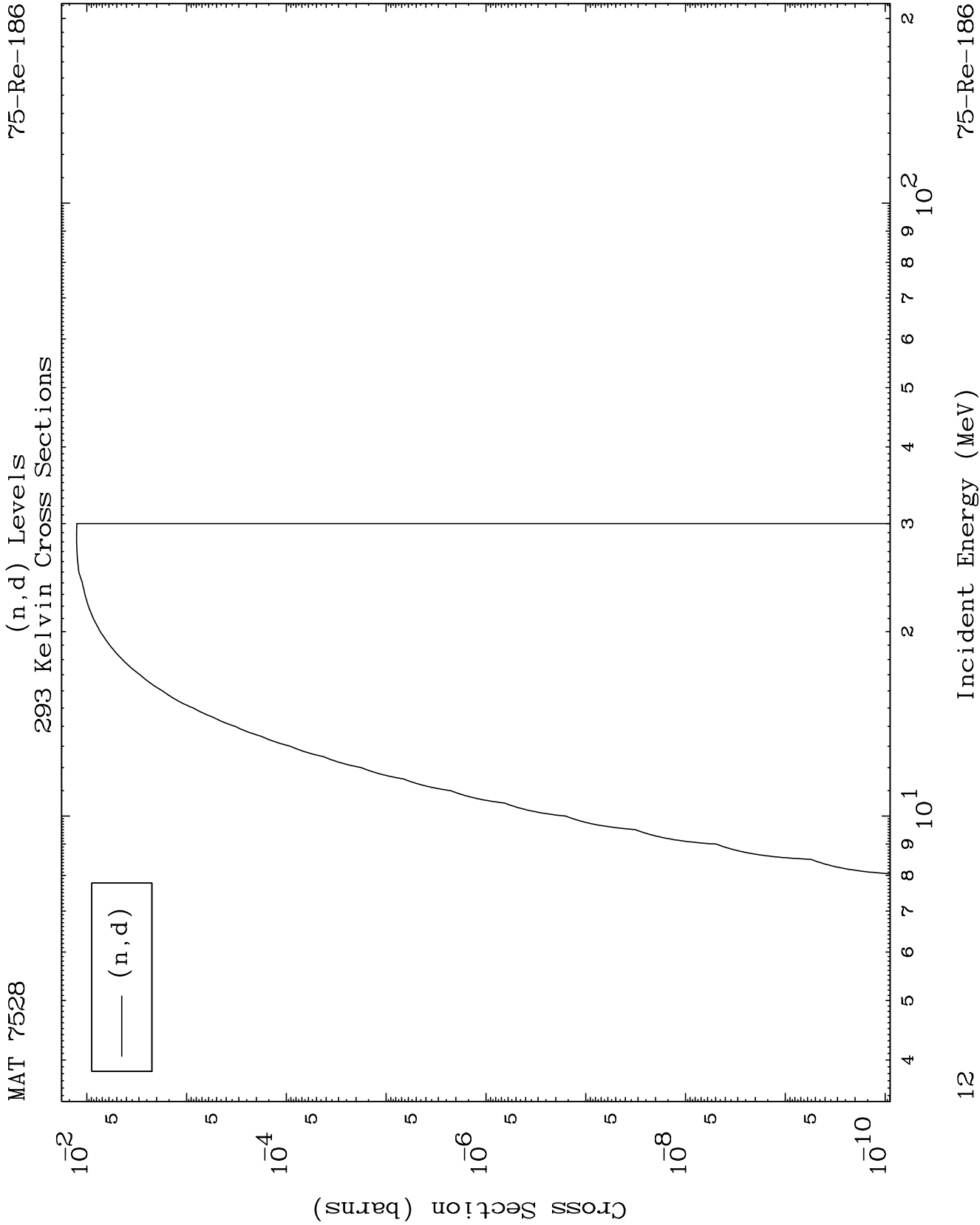








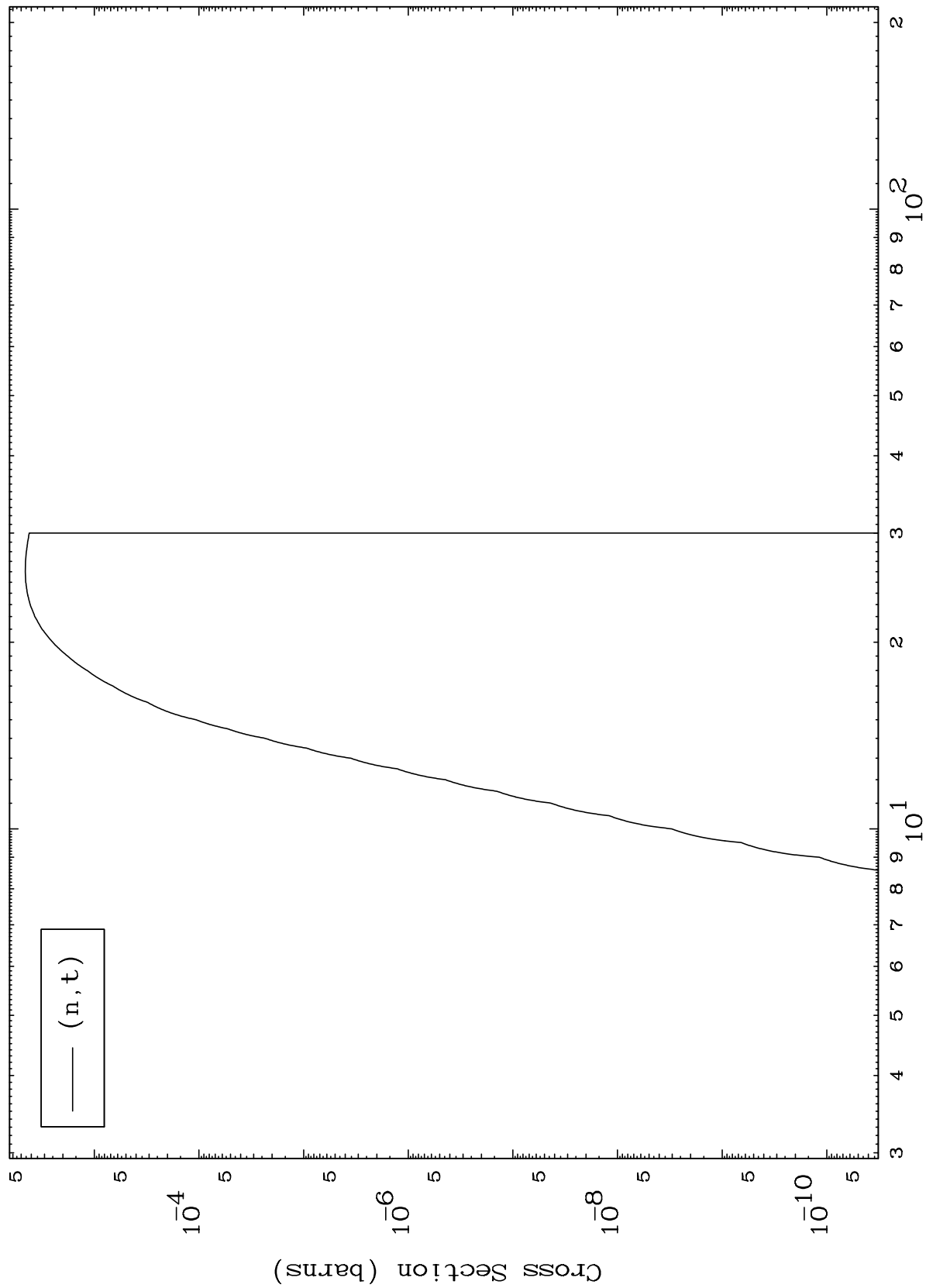




MAT 7528

75-Re-186

(n,t) Levels
293 Kelvin Cross Sections



Incident Energy (MeV)

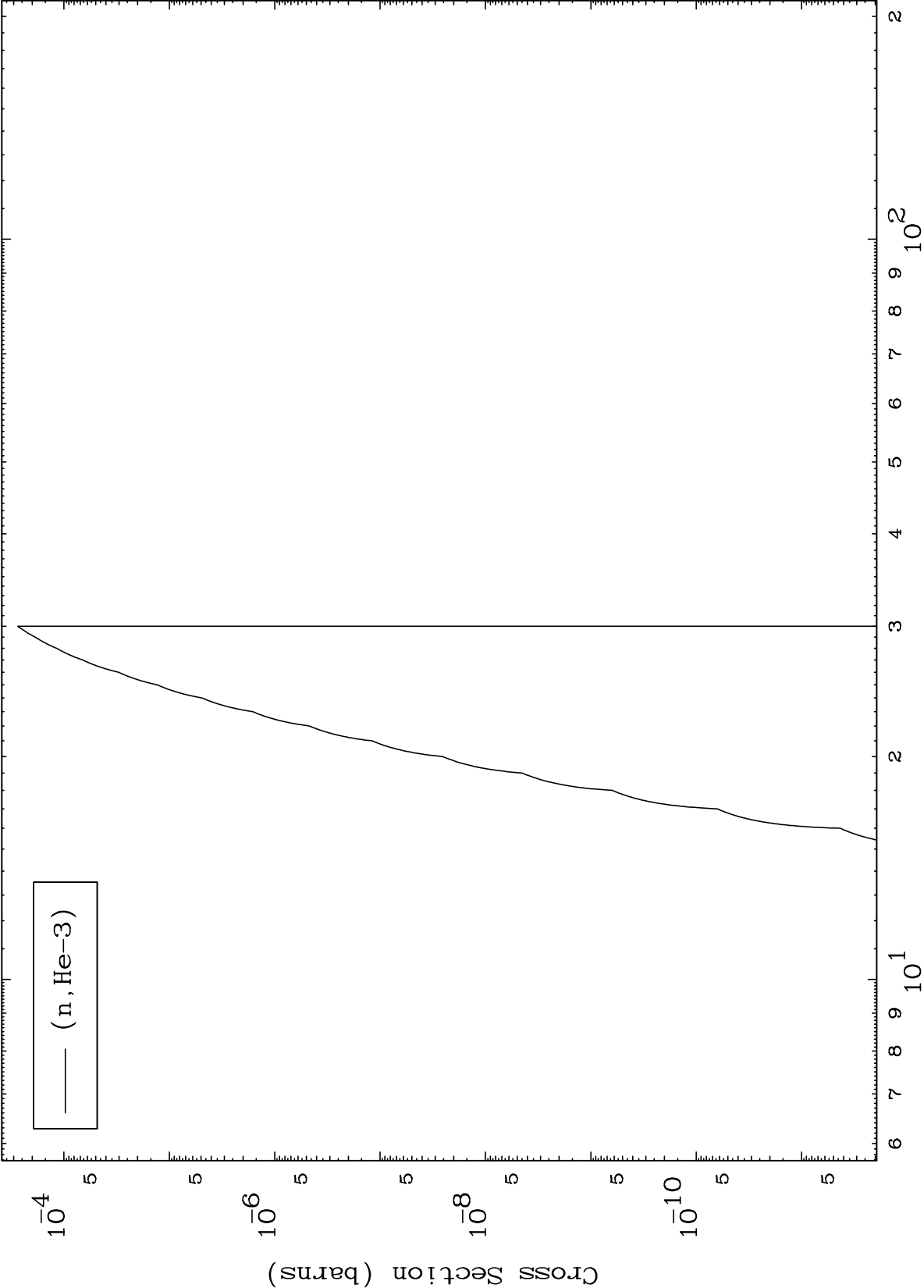
75-Re-186

13

MAT 7528

(n,He3) Levels
293 Kelvin Cross Sections

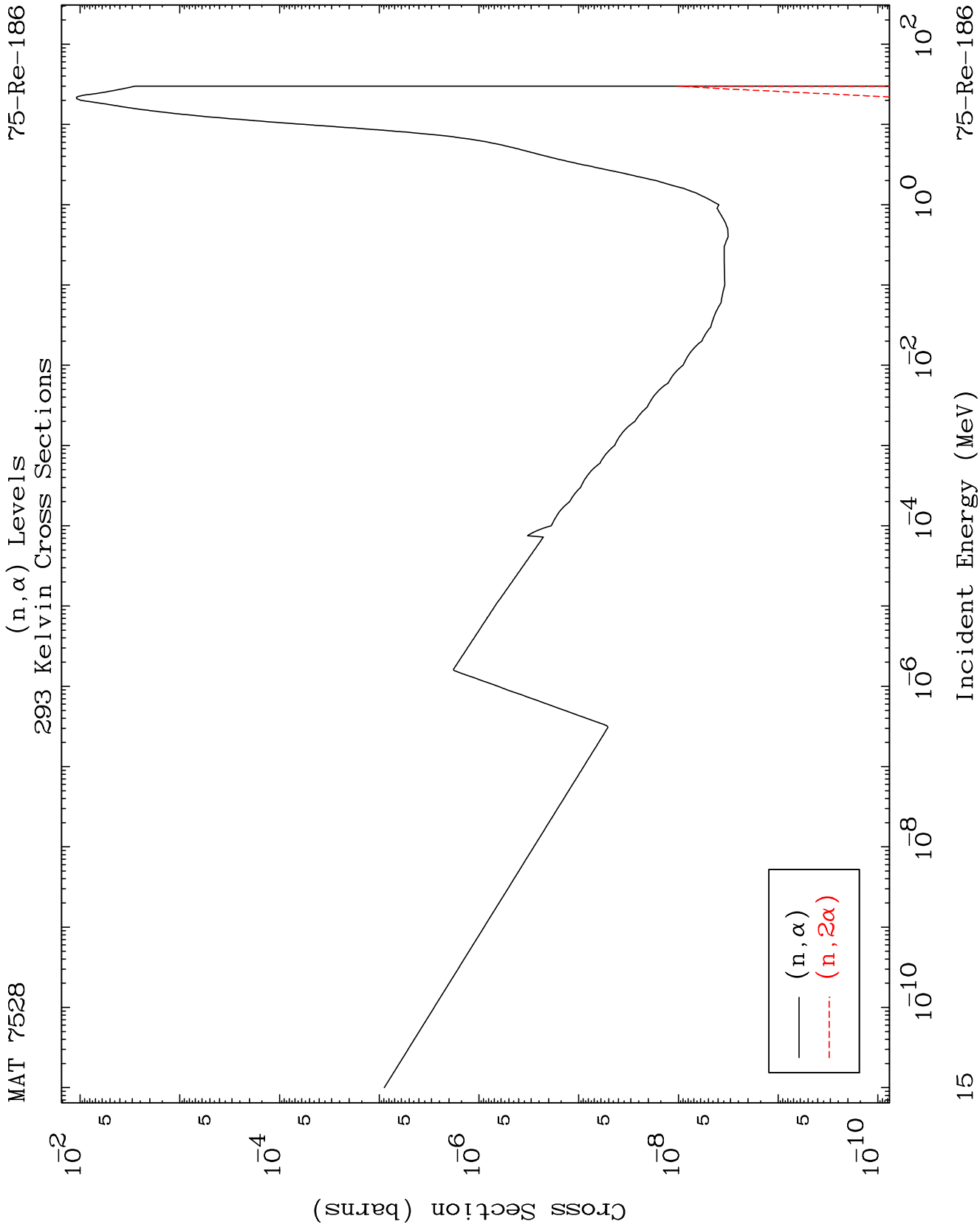
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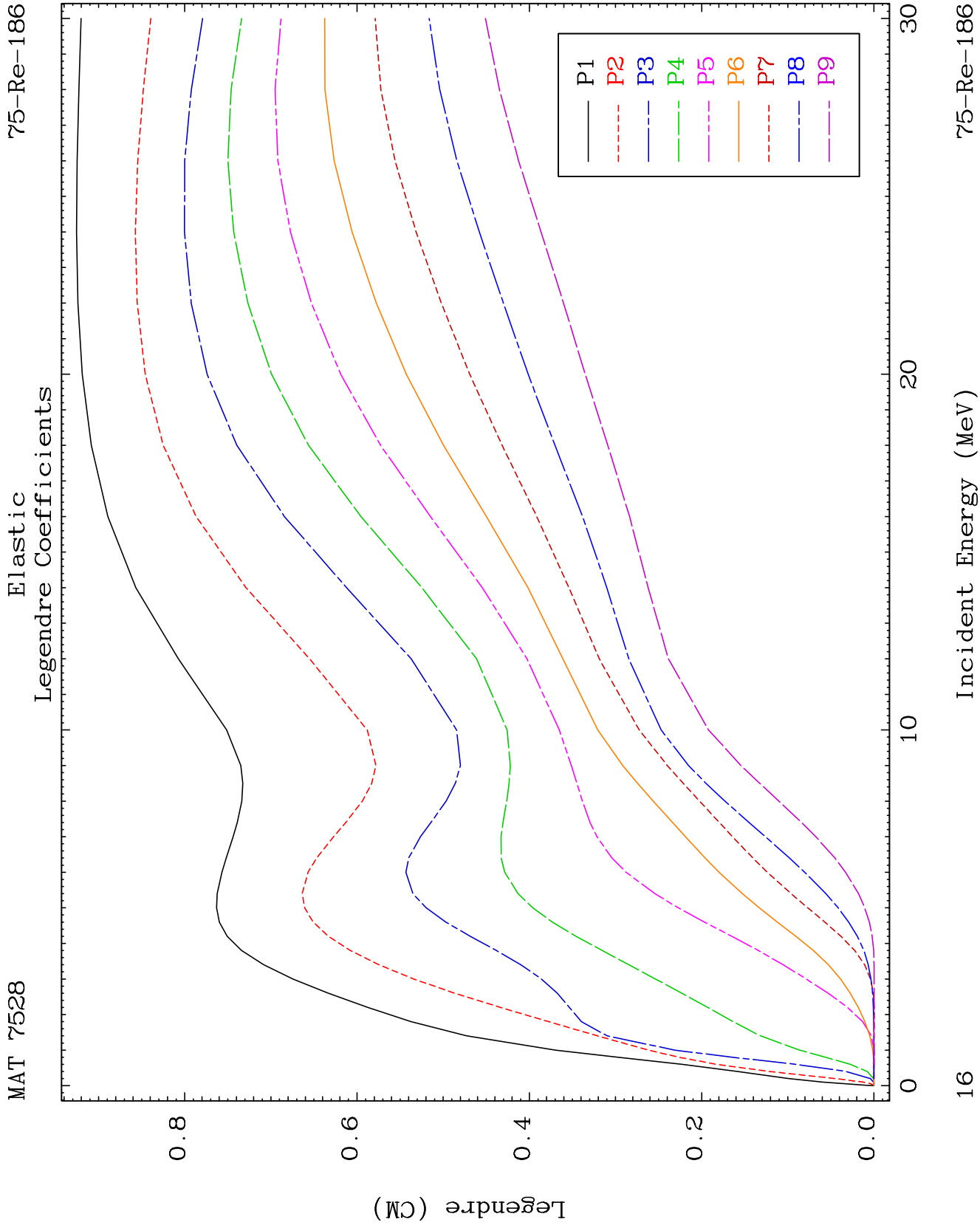


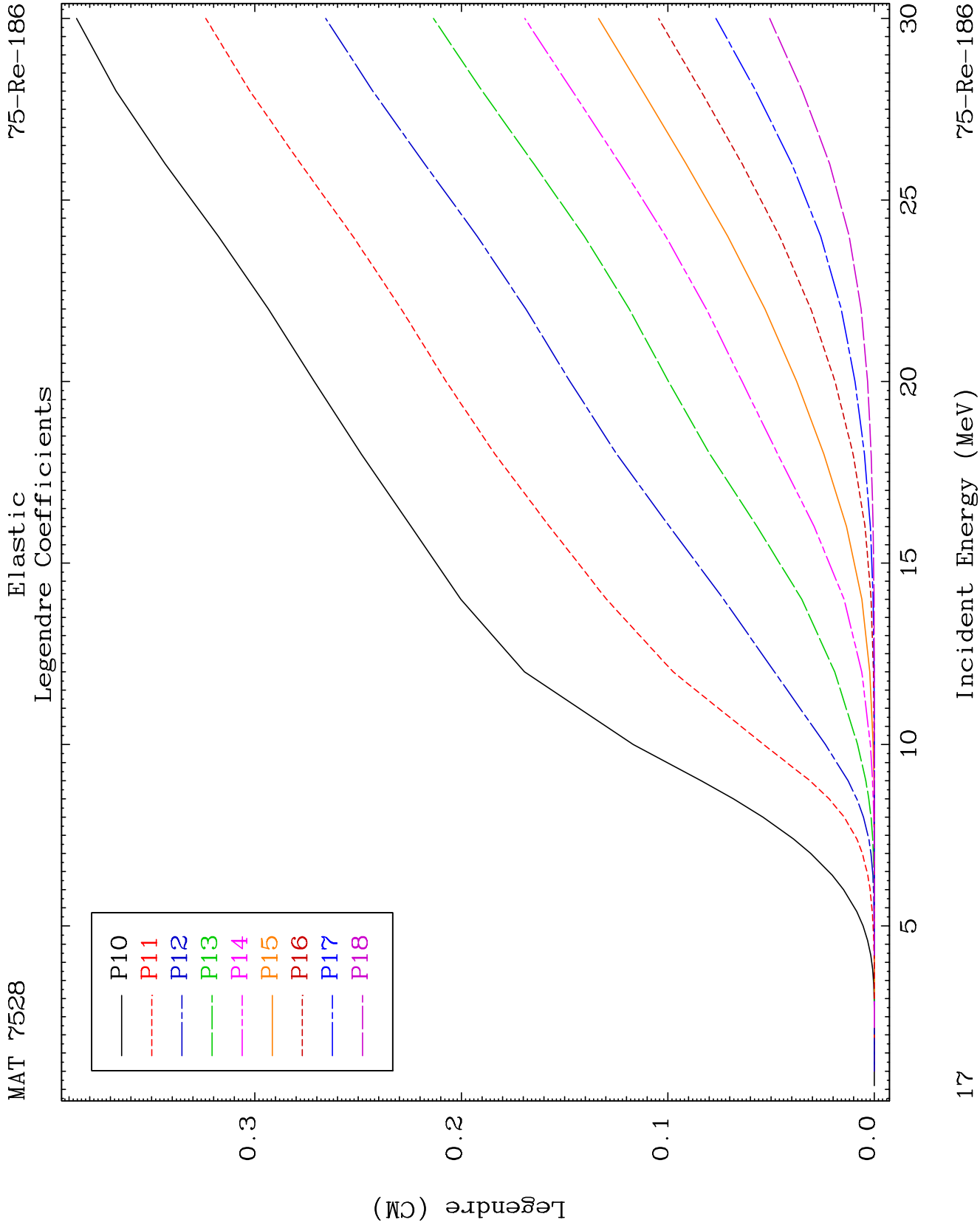
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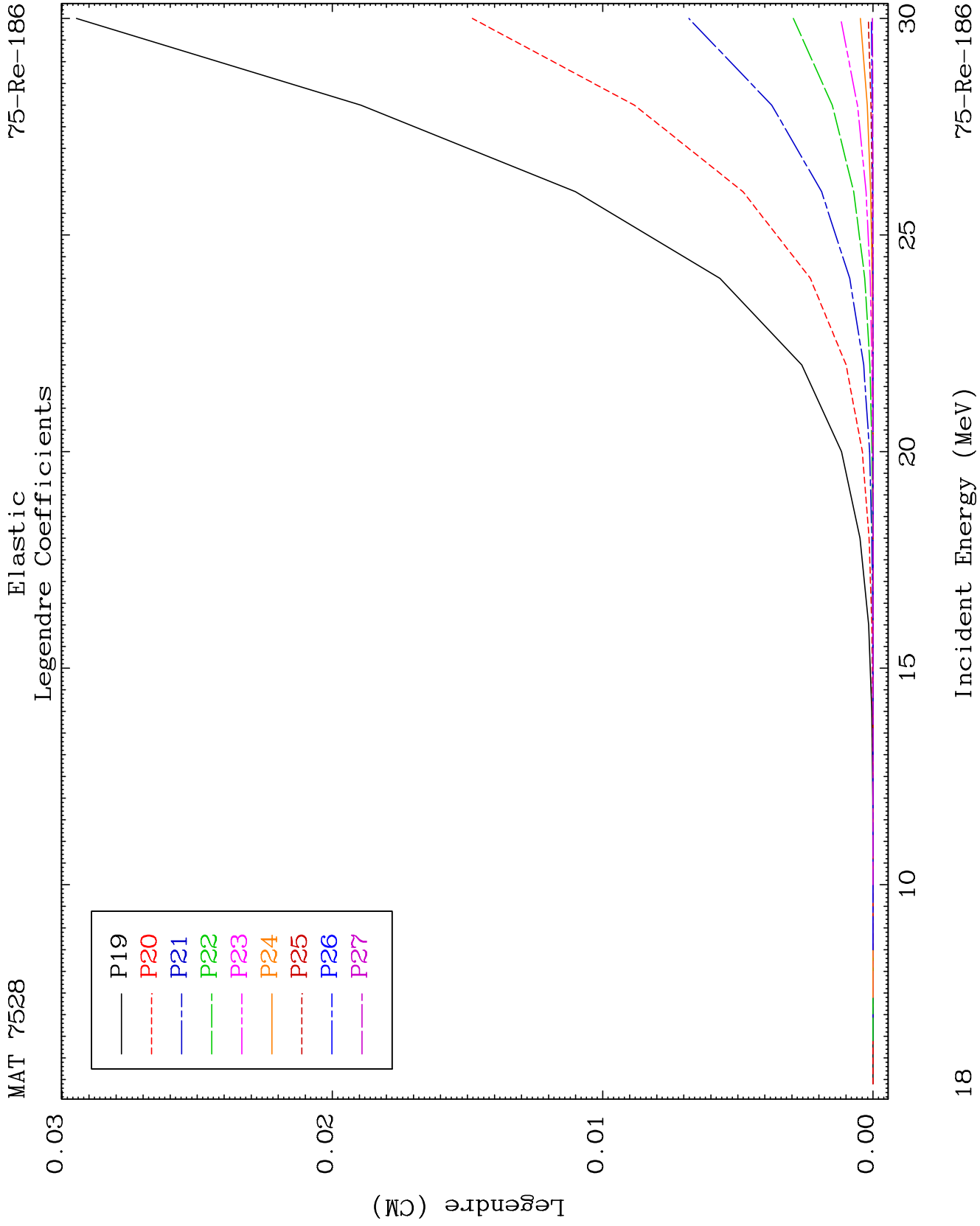
Incident Energy (MeV)

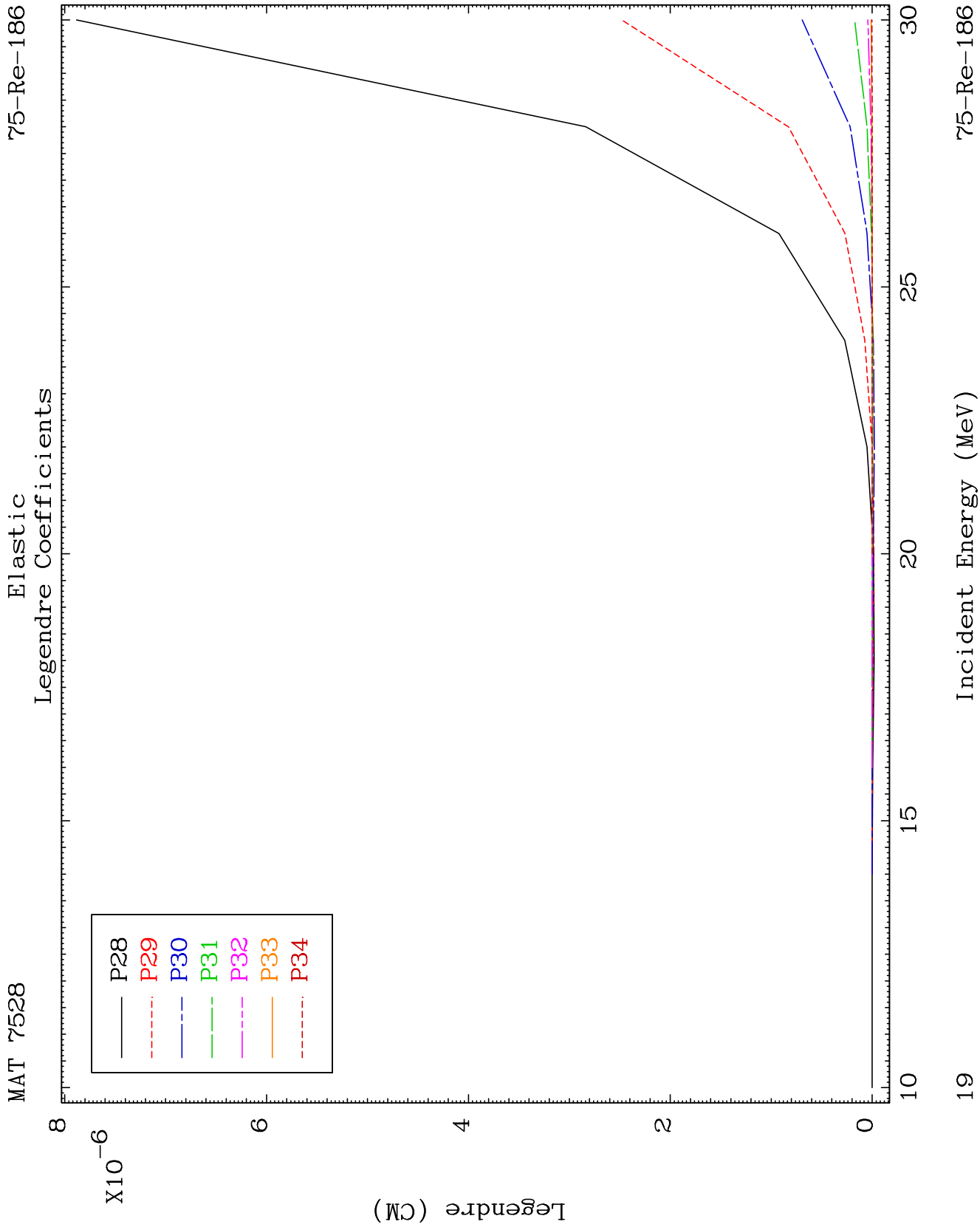
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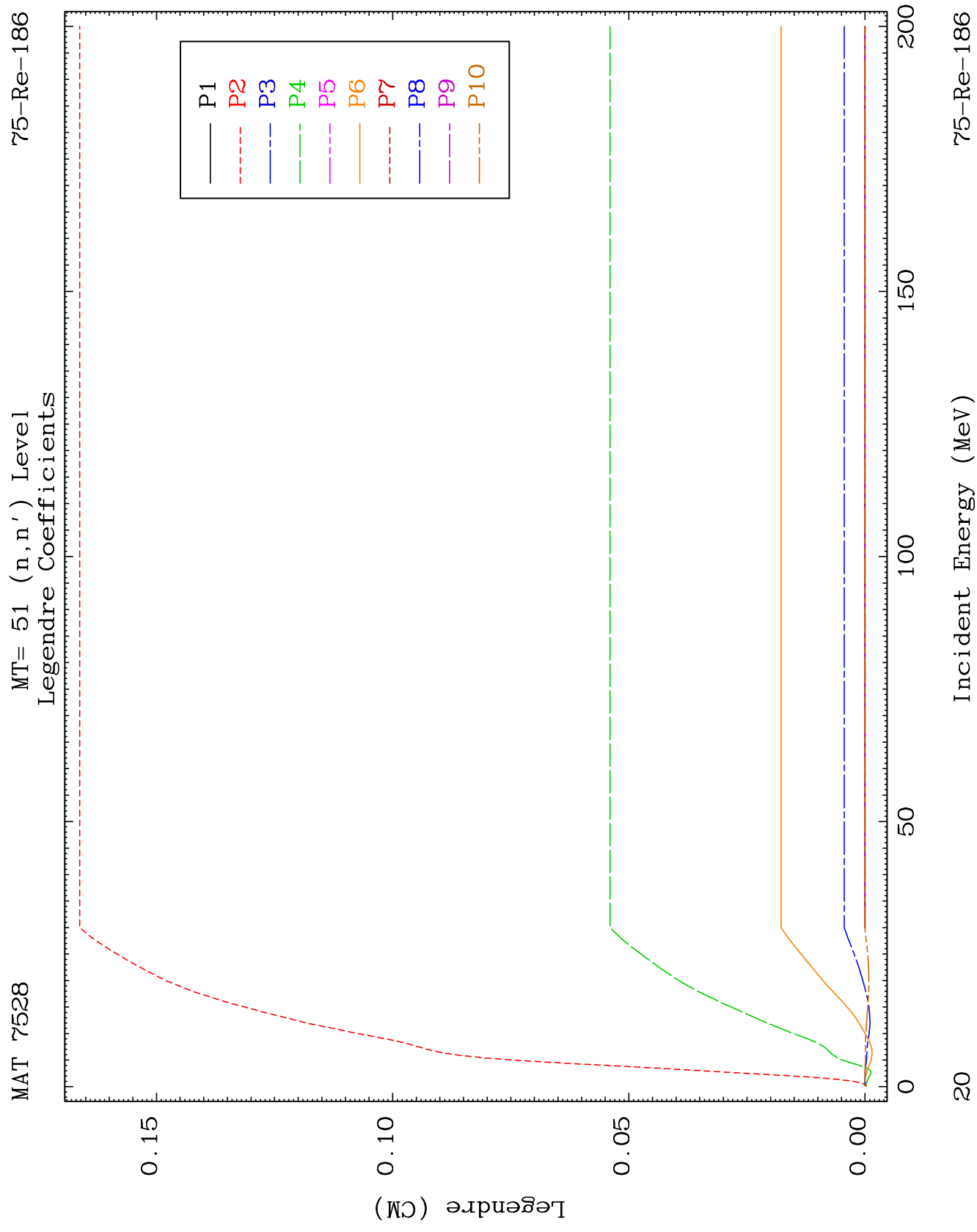


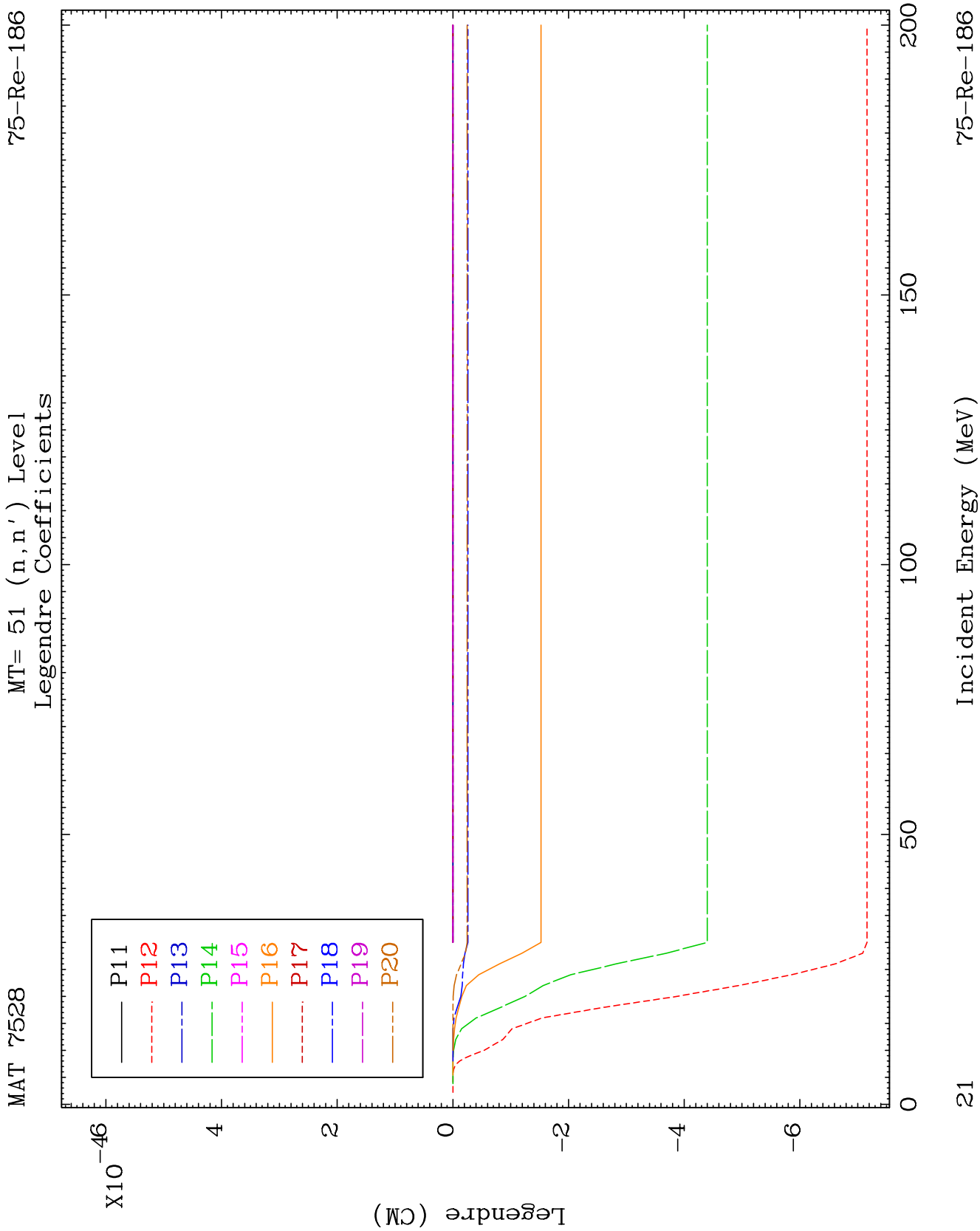


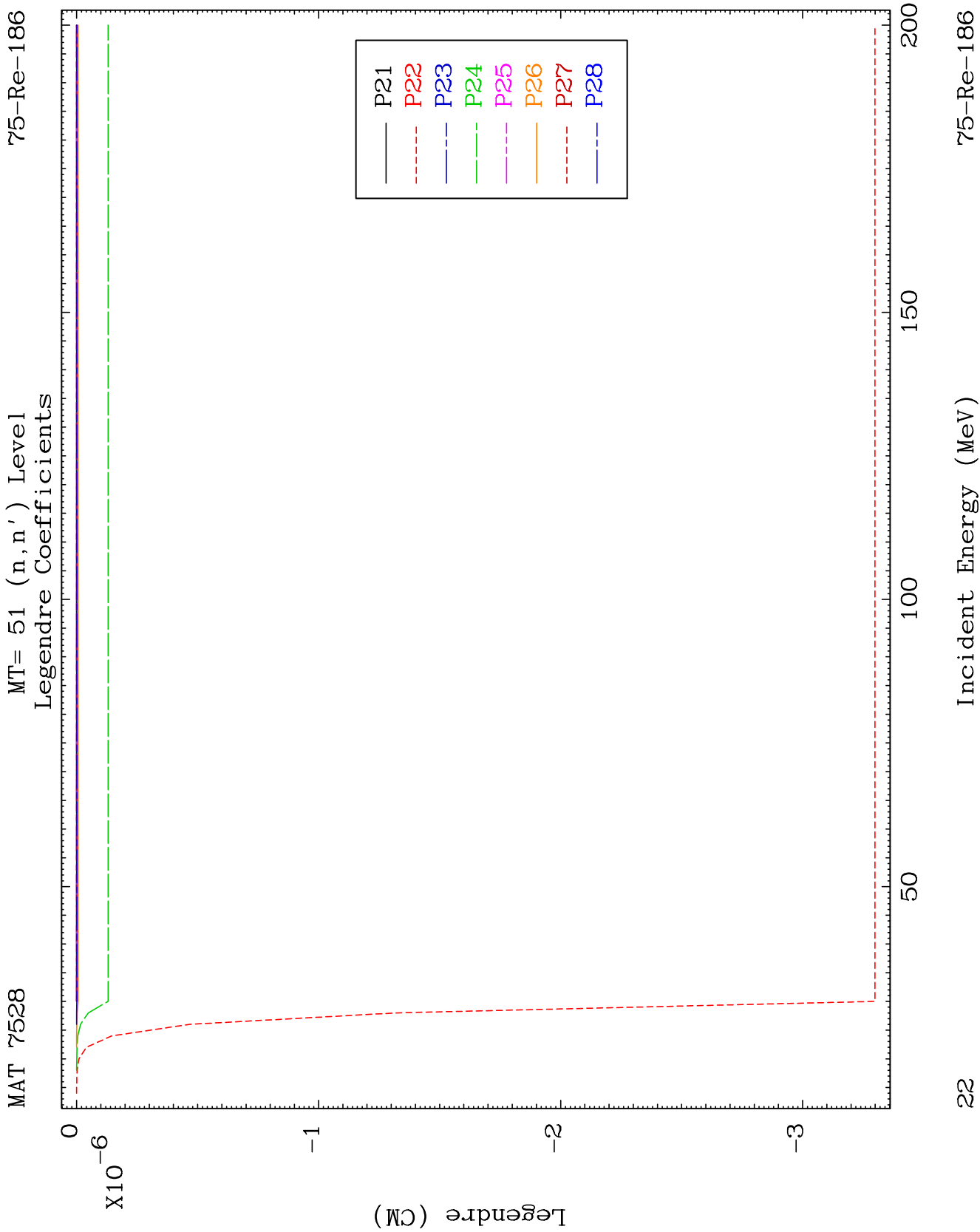








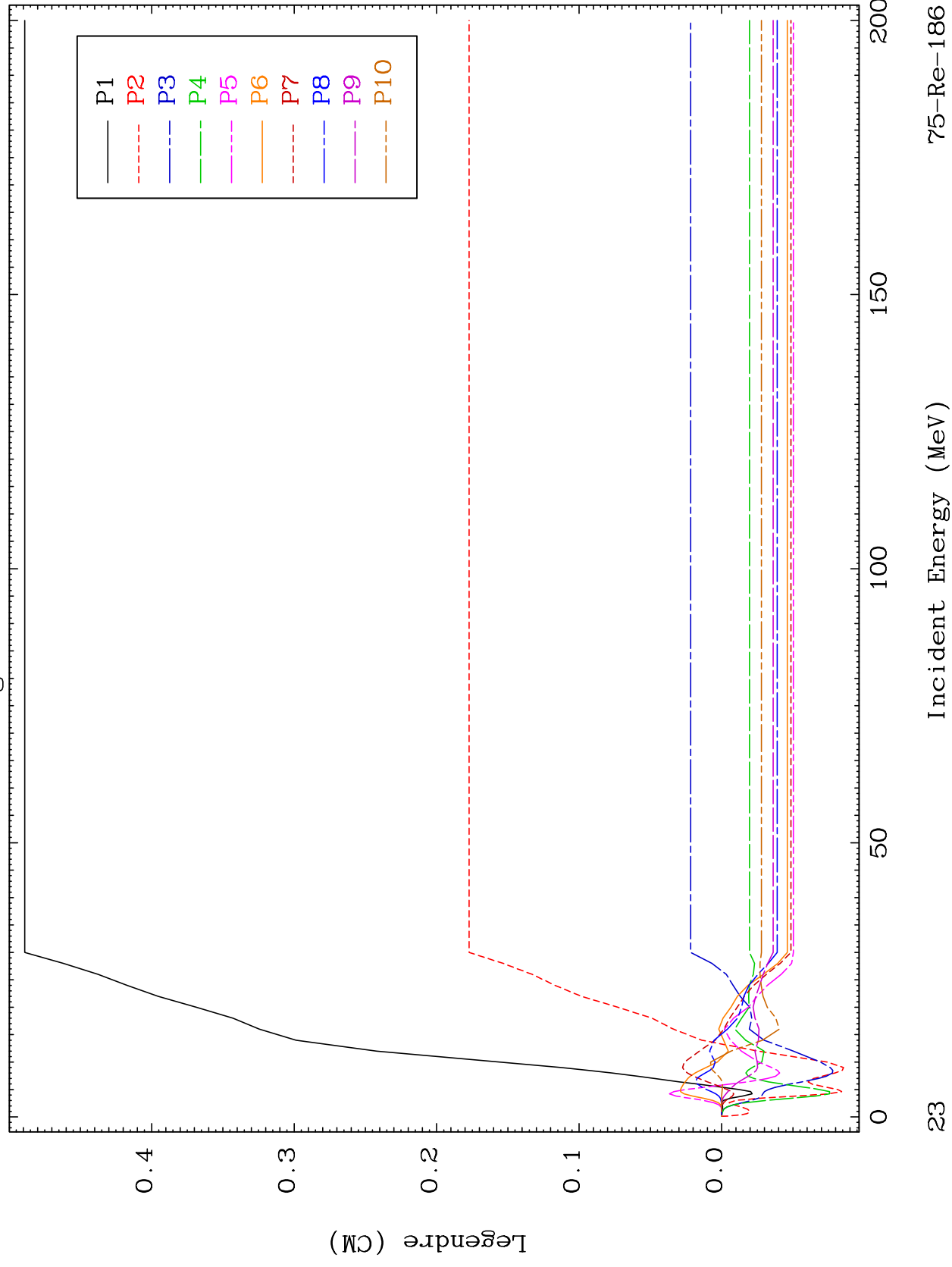


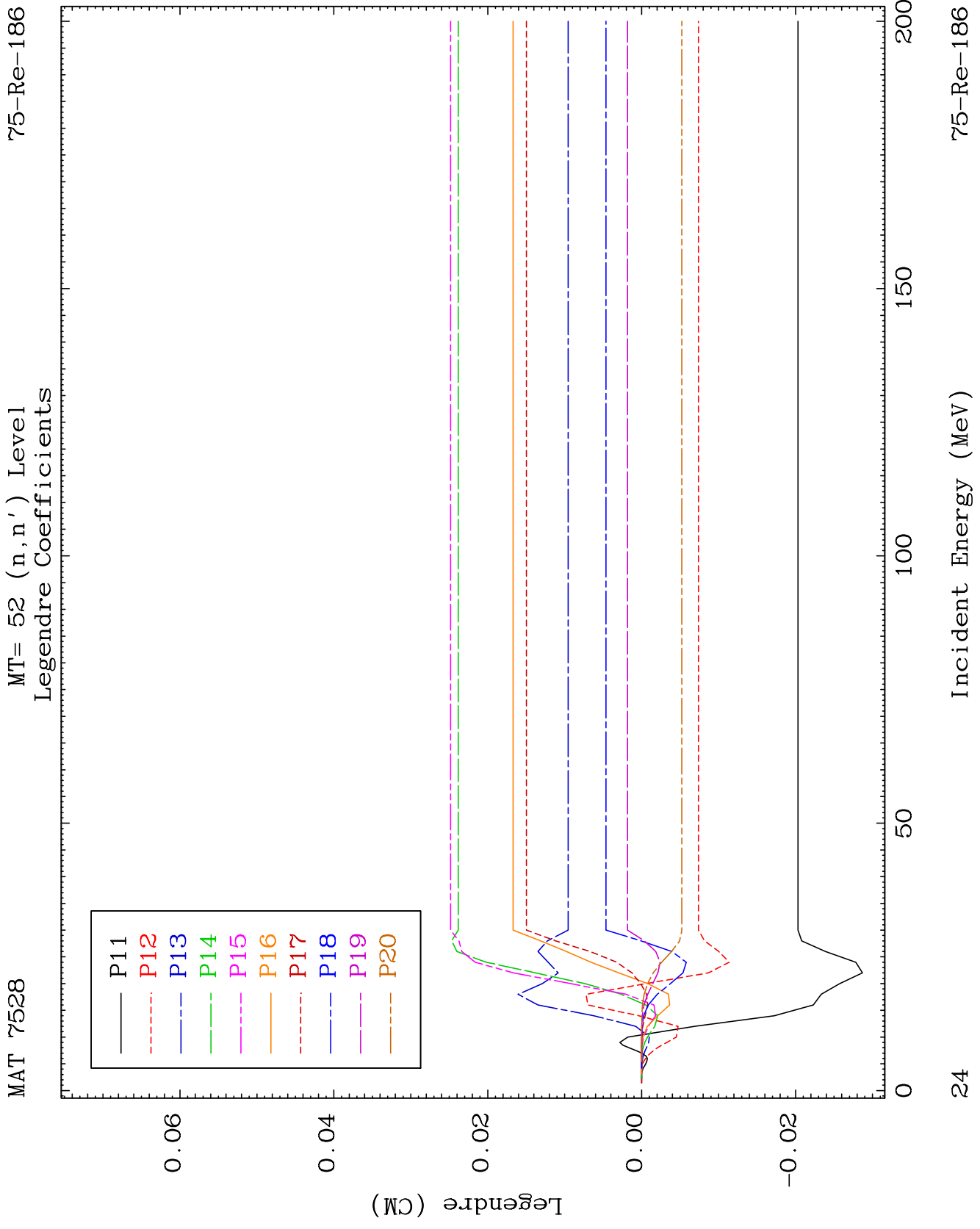


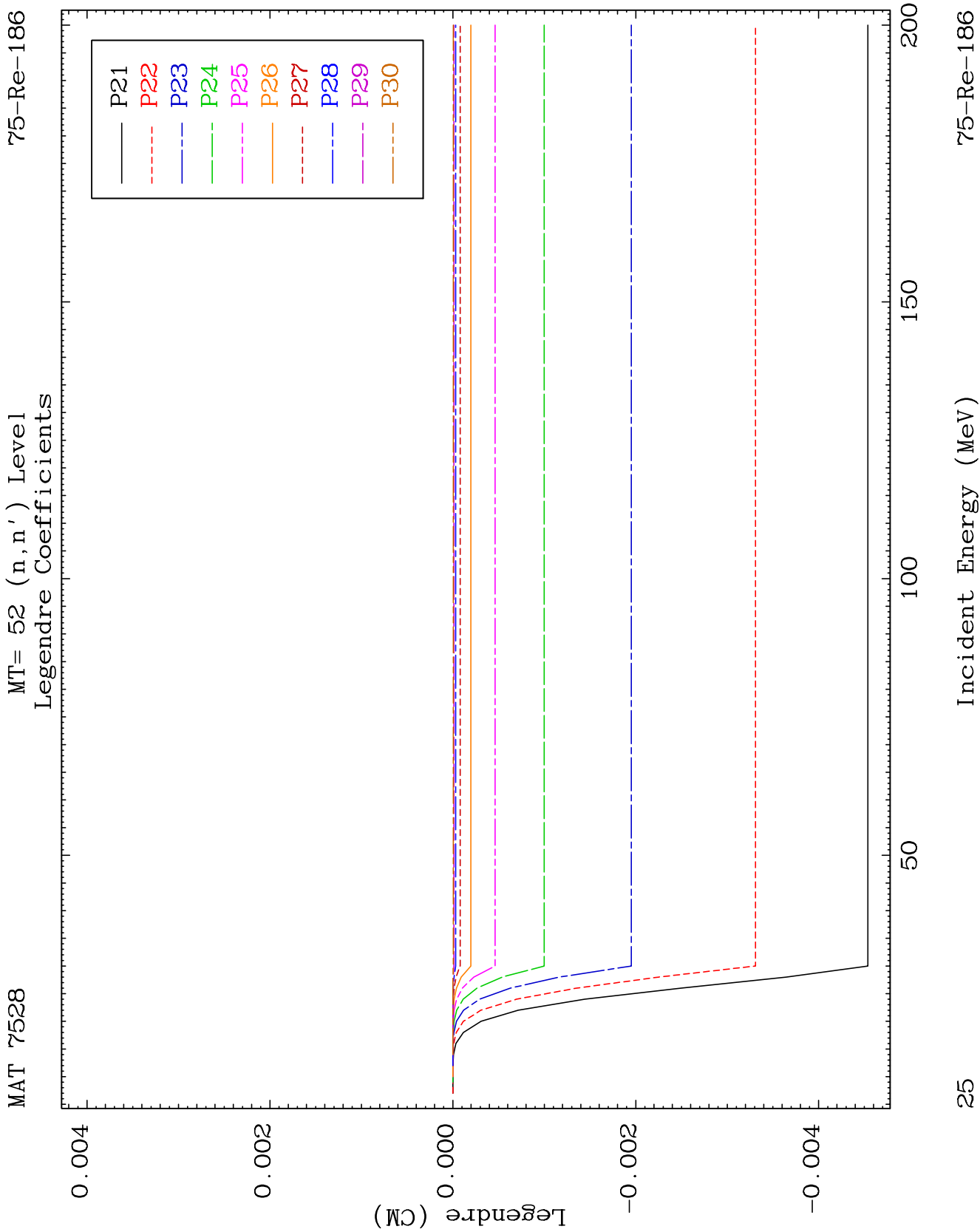
MAT 7528

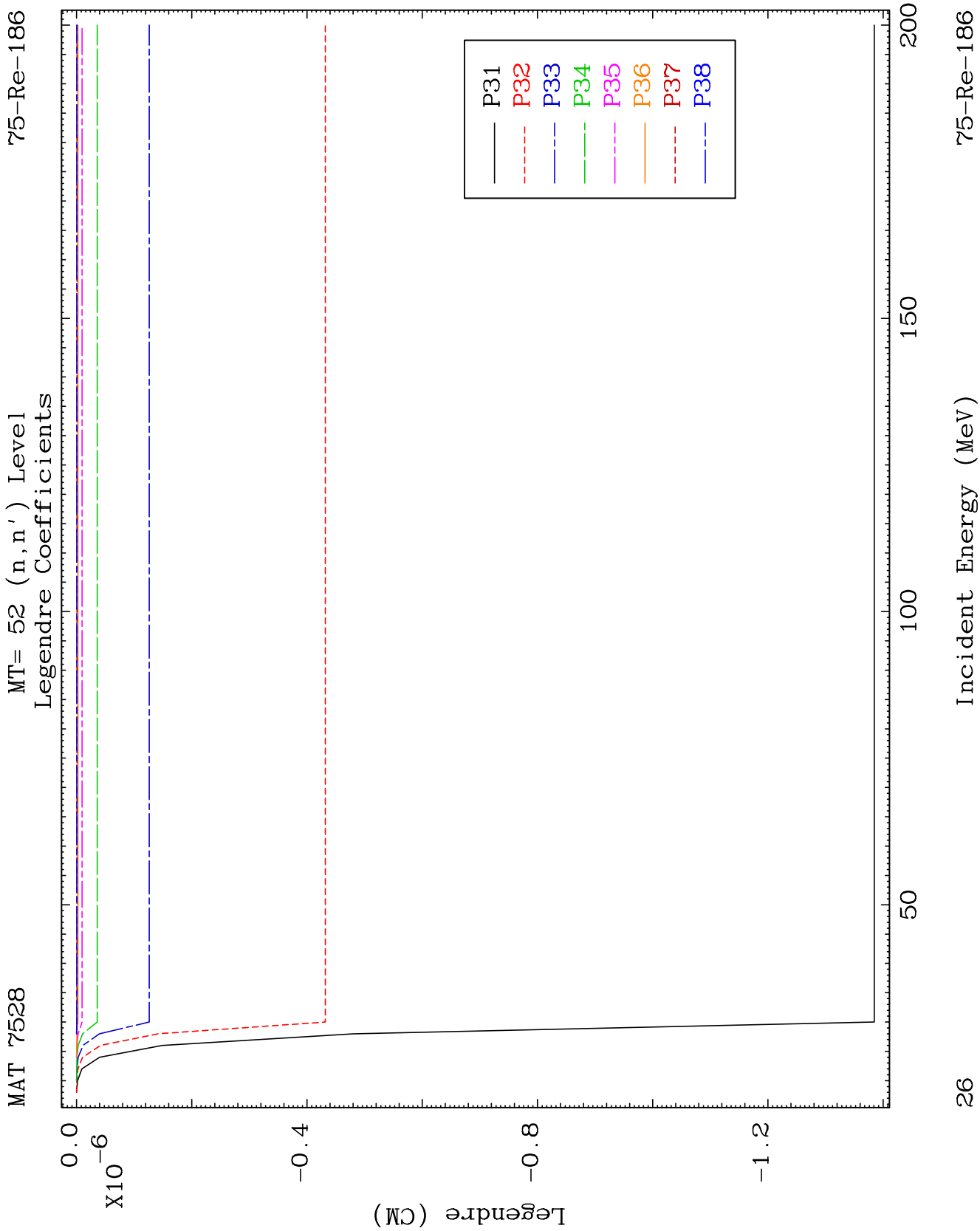
MT= 52 (n, n') Level
Legendre Coefficients

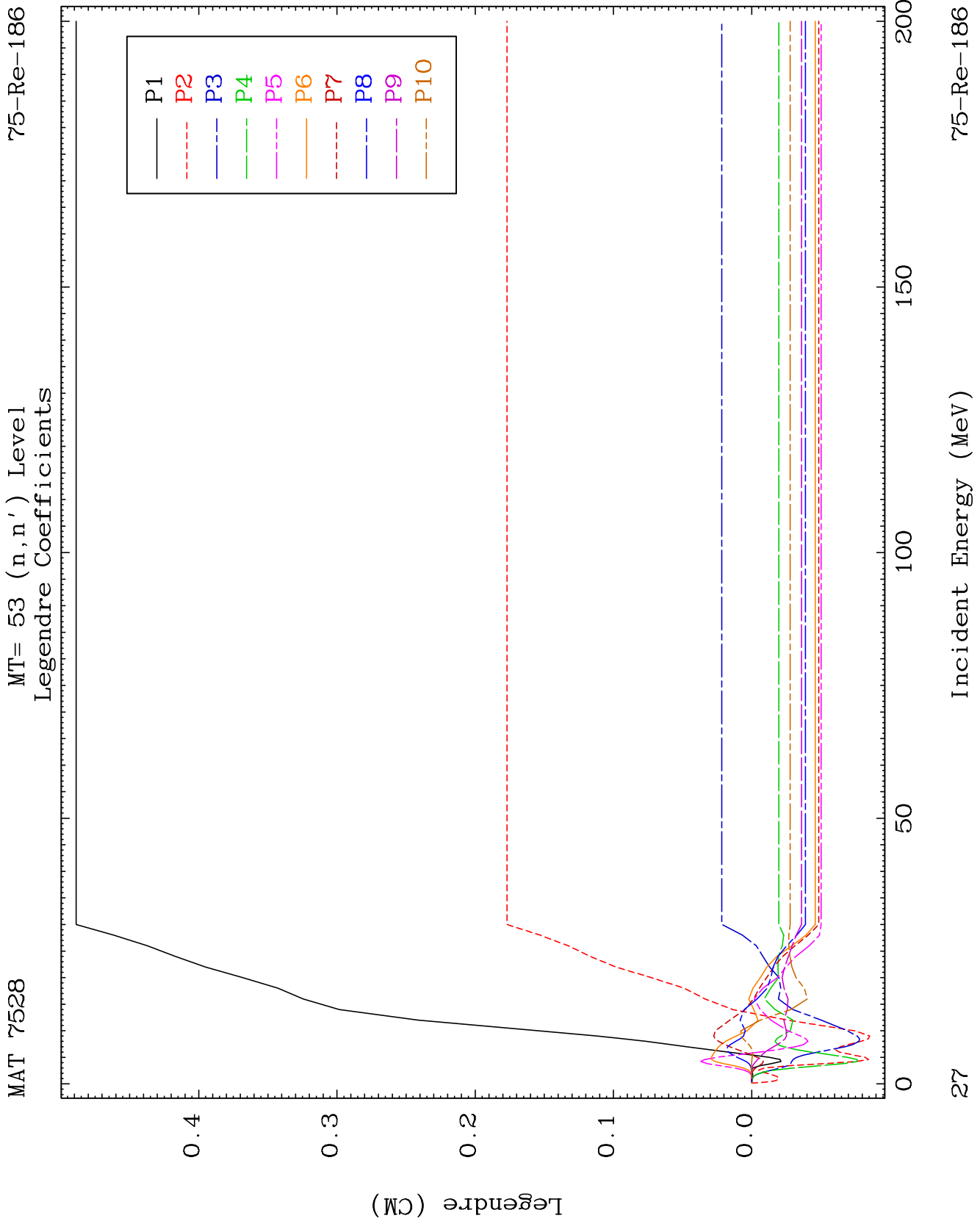
75-Re-186







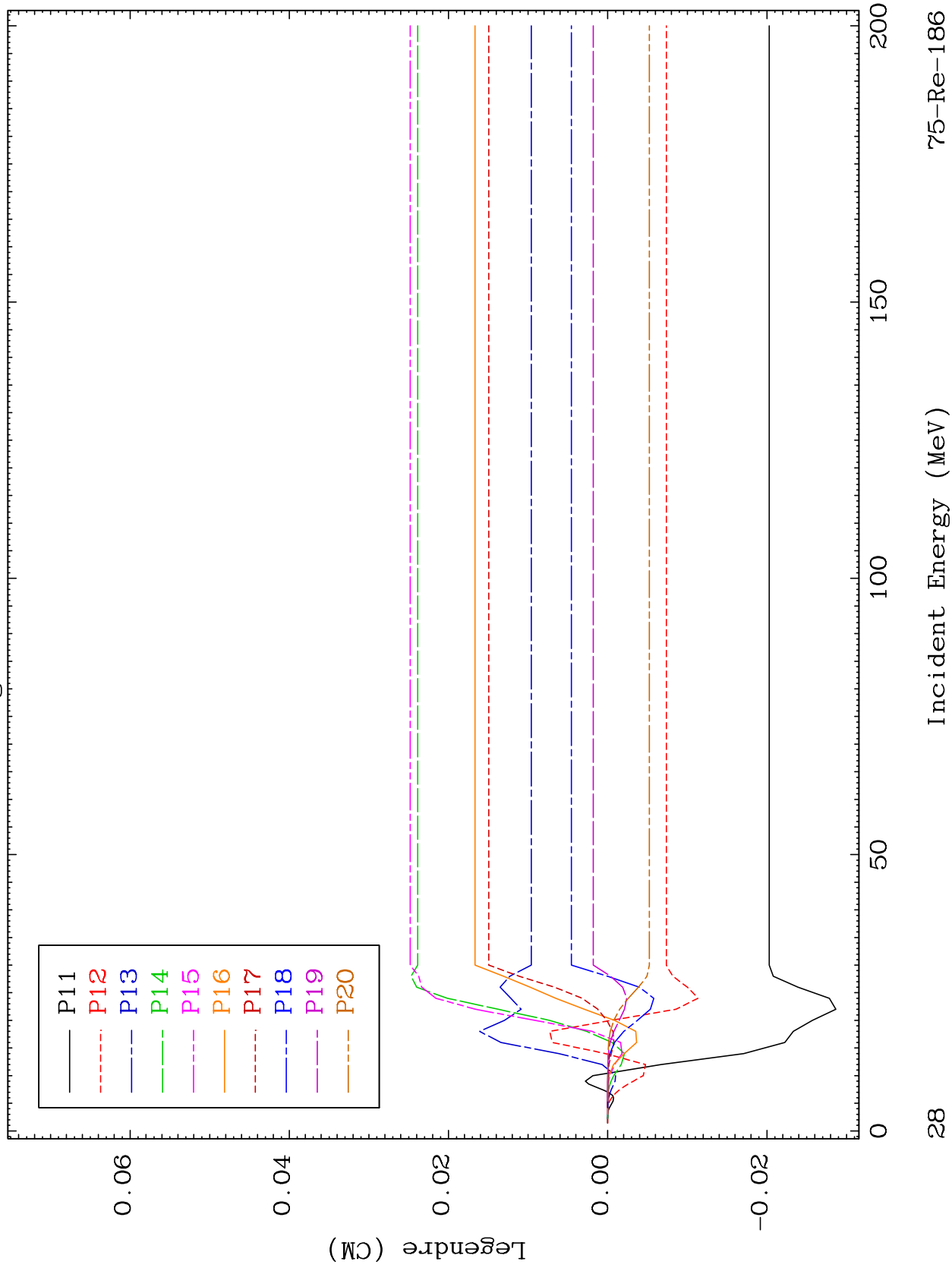


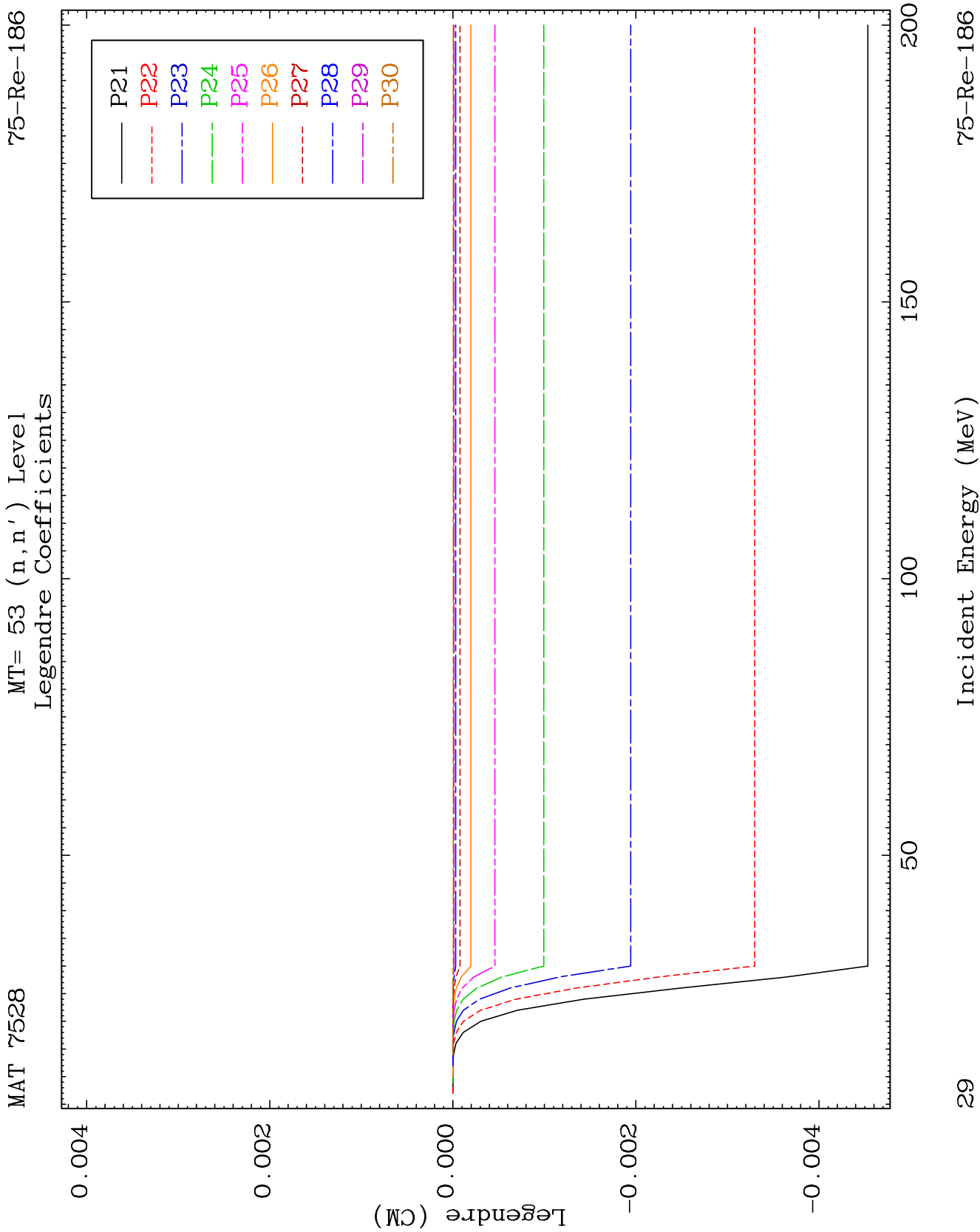


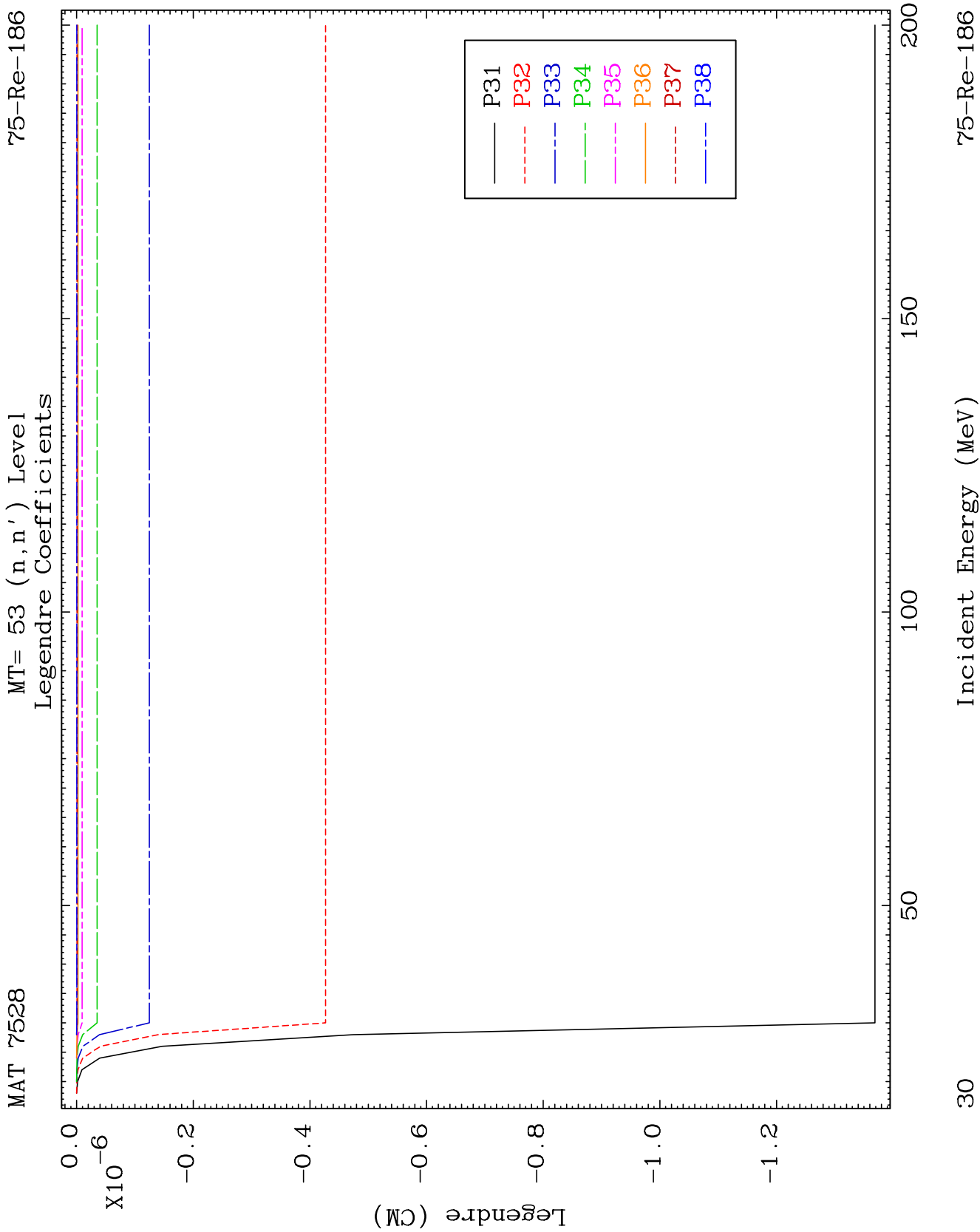
MAT 7528

MT= 53 (n, n') Level
Legendre Coefficients

75-Re-186



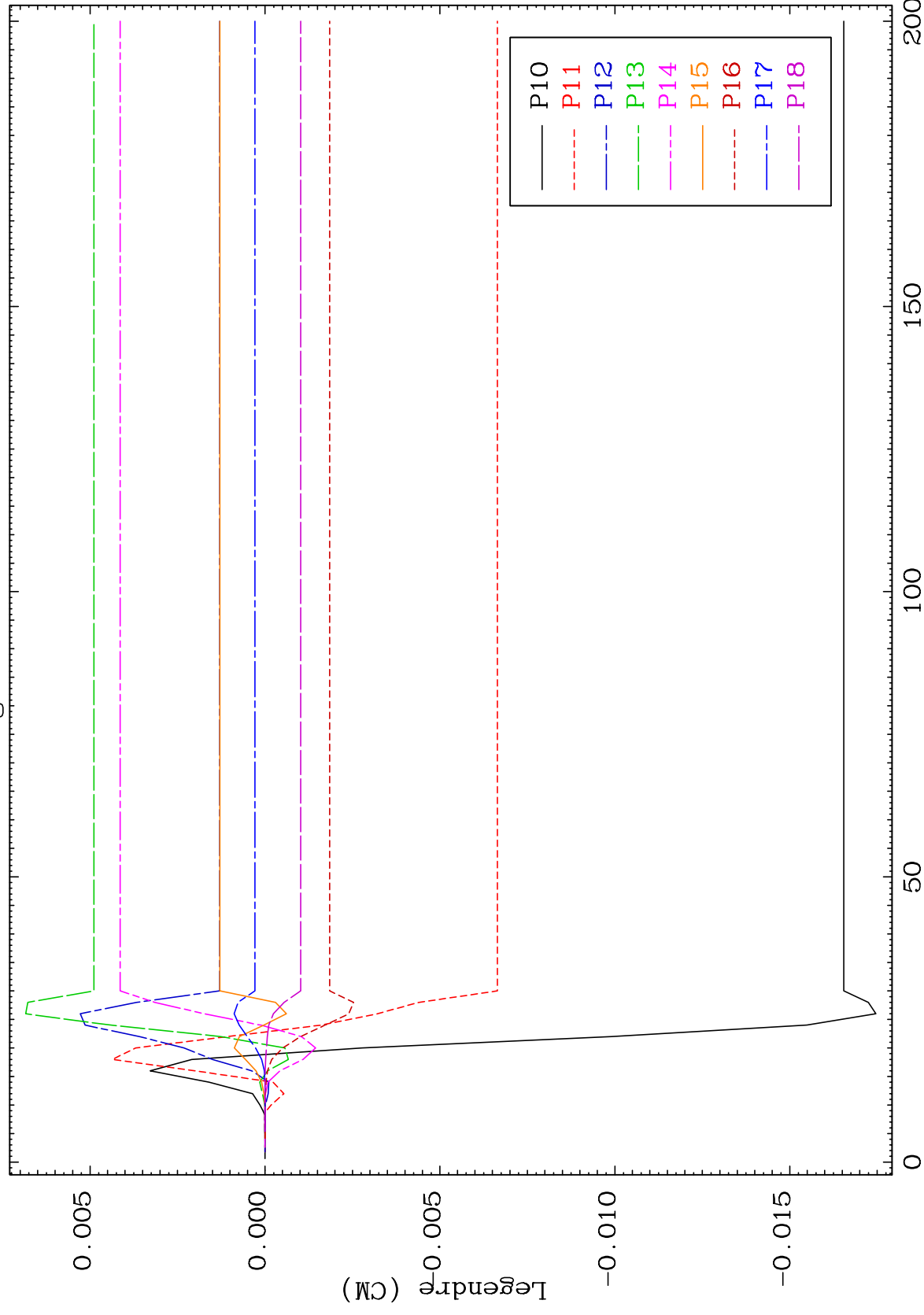




MAT 7528

MT= 54 (n, n') Level
Legendre Coefficients

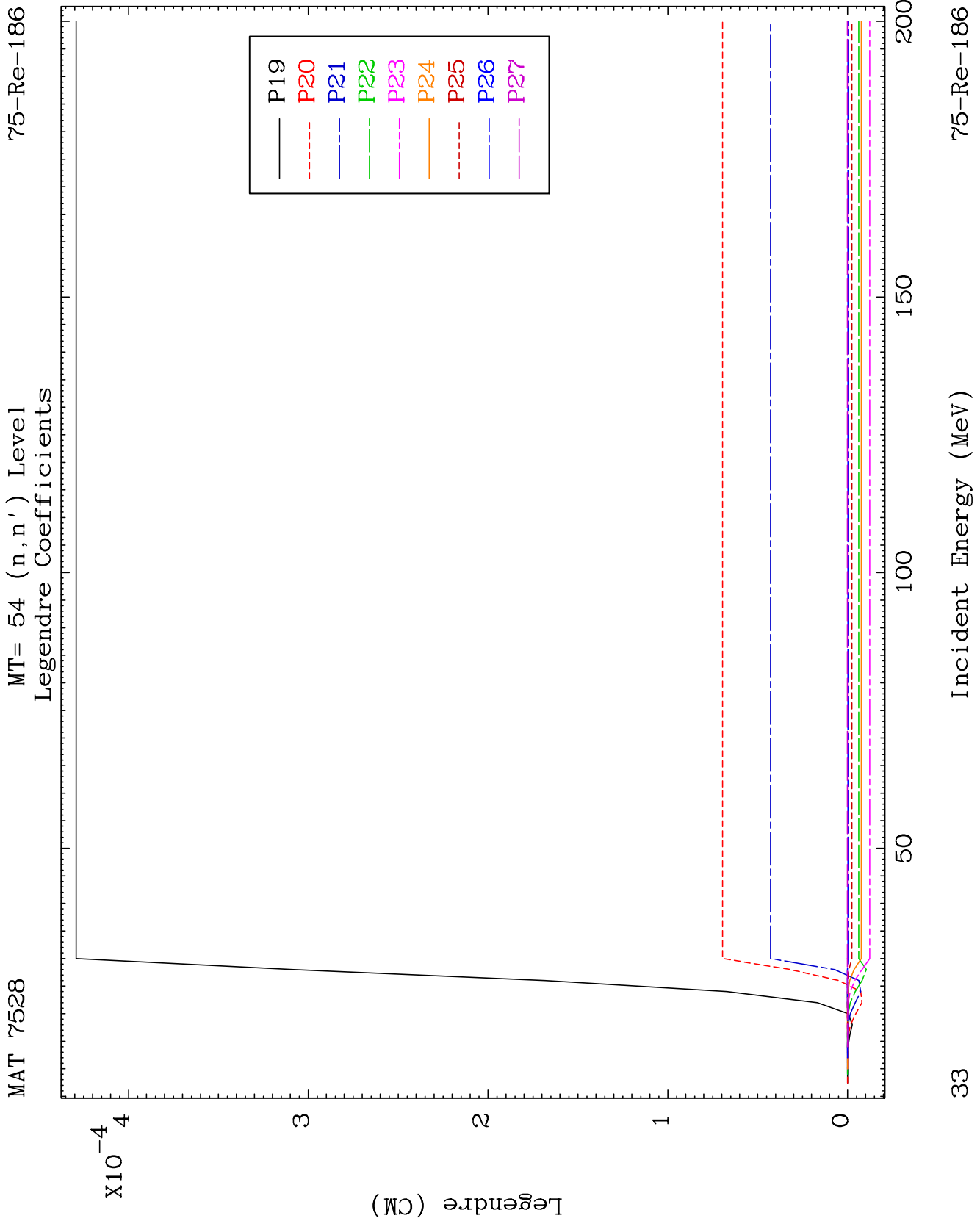
75-Re-186



23

Incident Energy (MeV)

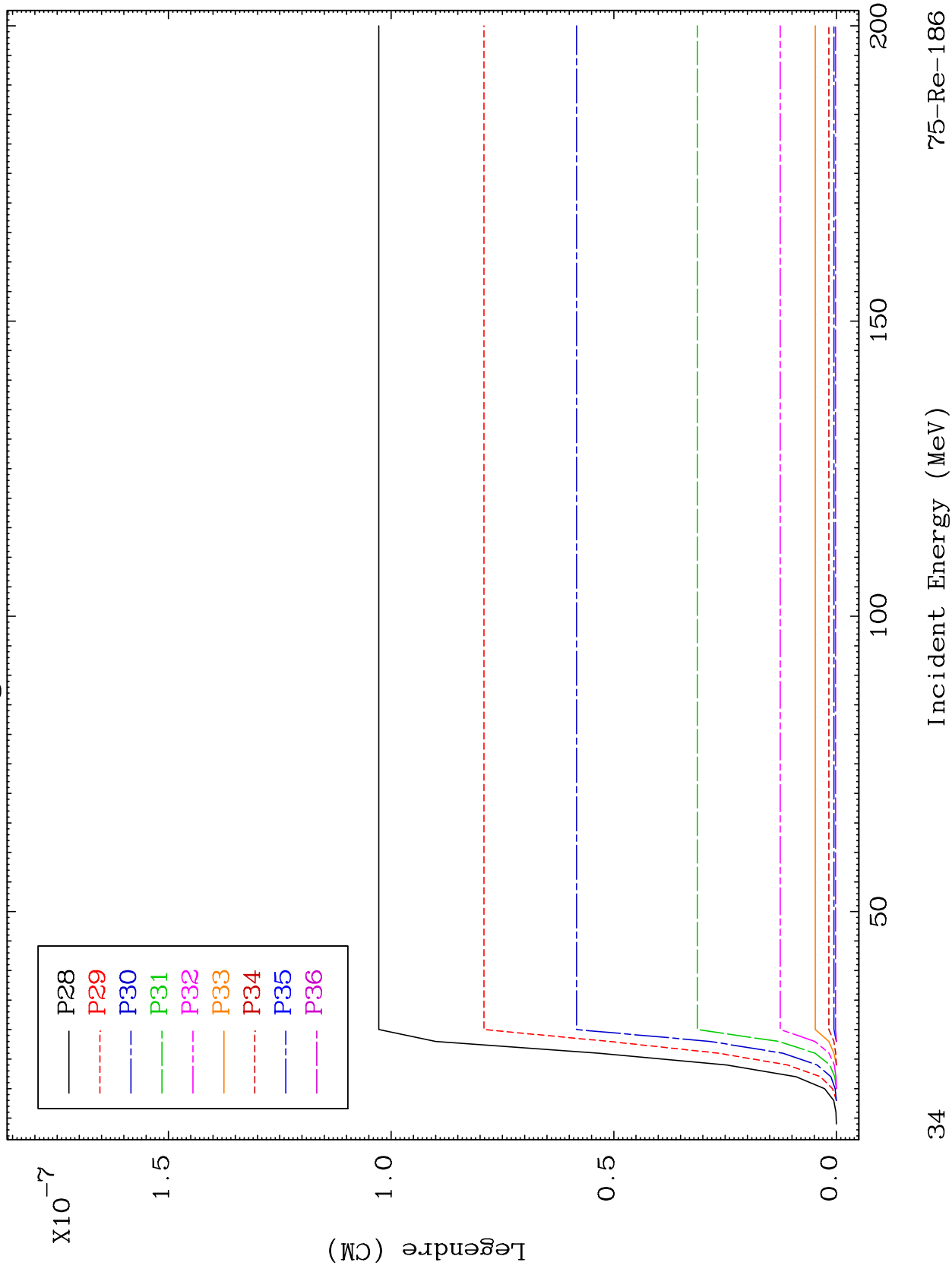
75-Re-186



MAT 7528

MT= 54 (n, n') Level
Legendre Coefficients

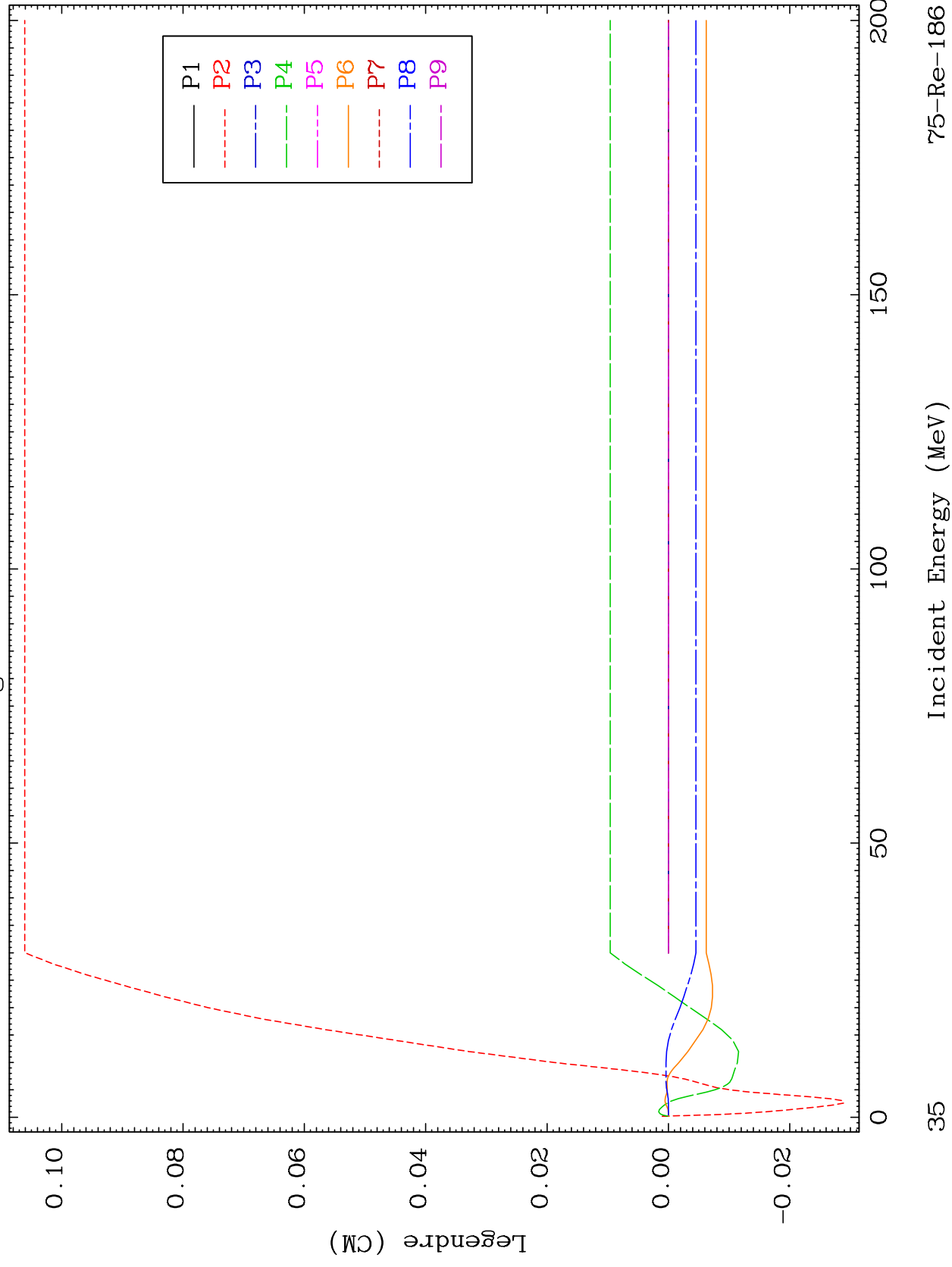
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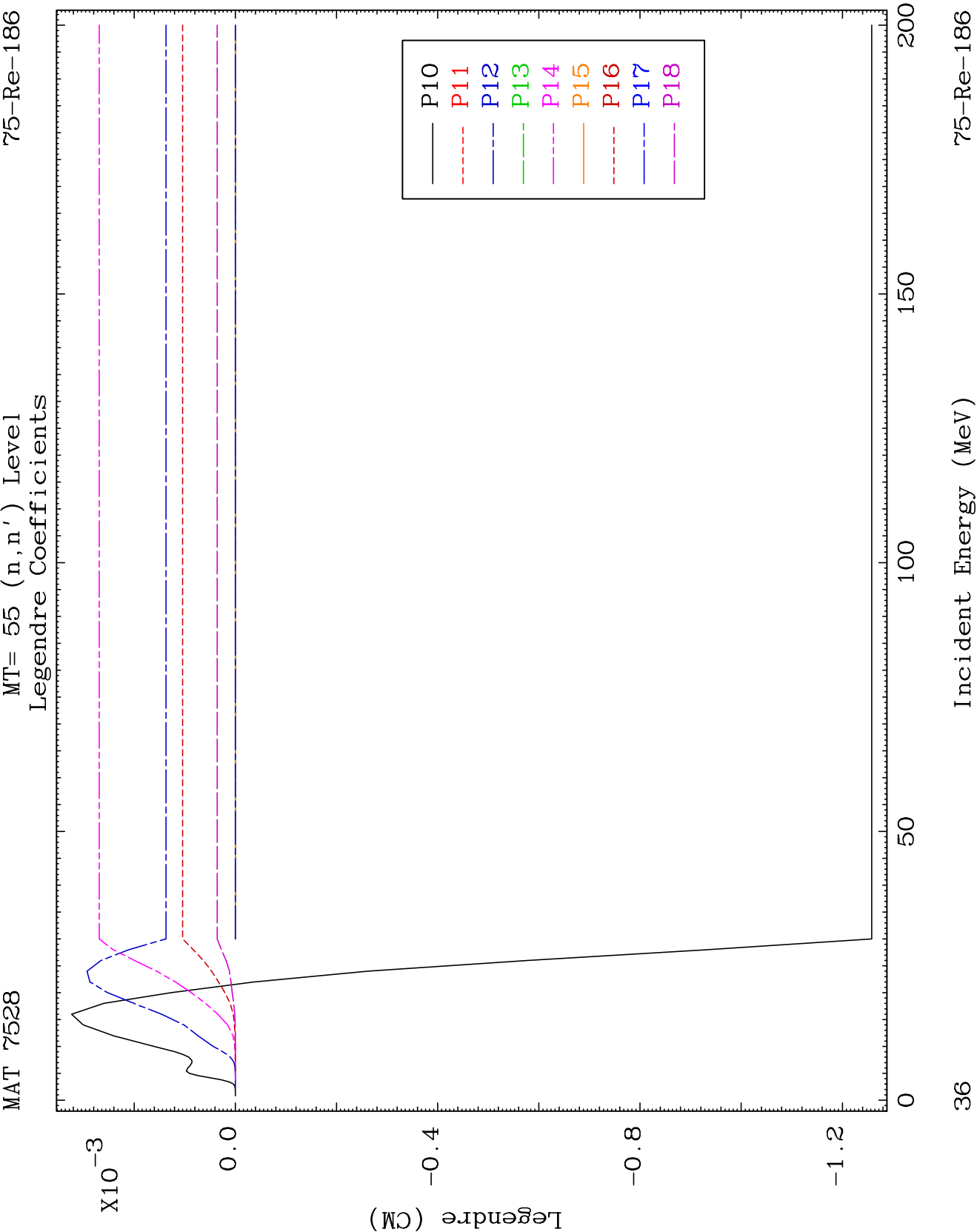


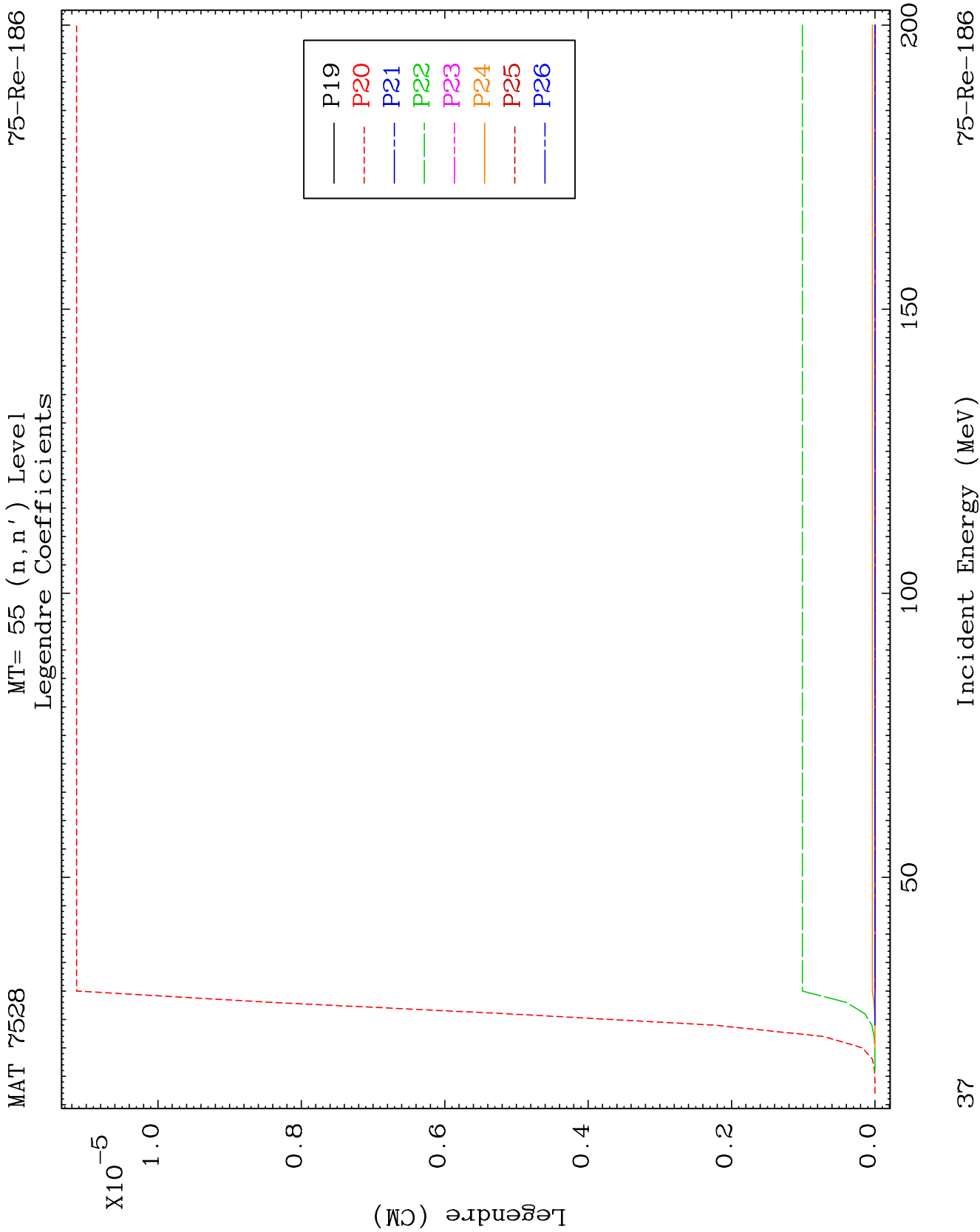
MAT 7528

MT= 55 (n, n') Level
Legendre Coefficients

75-Re-186



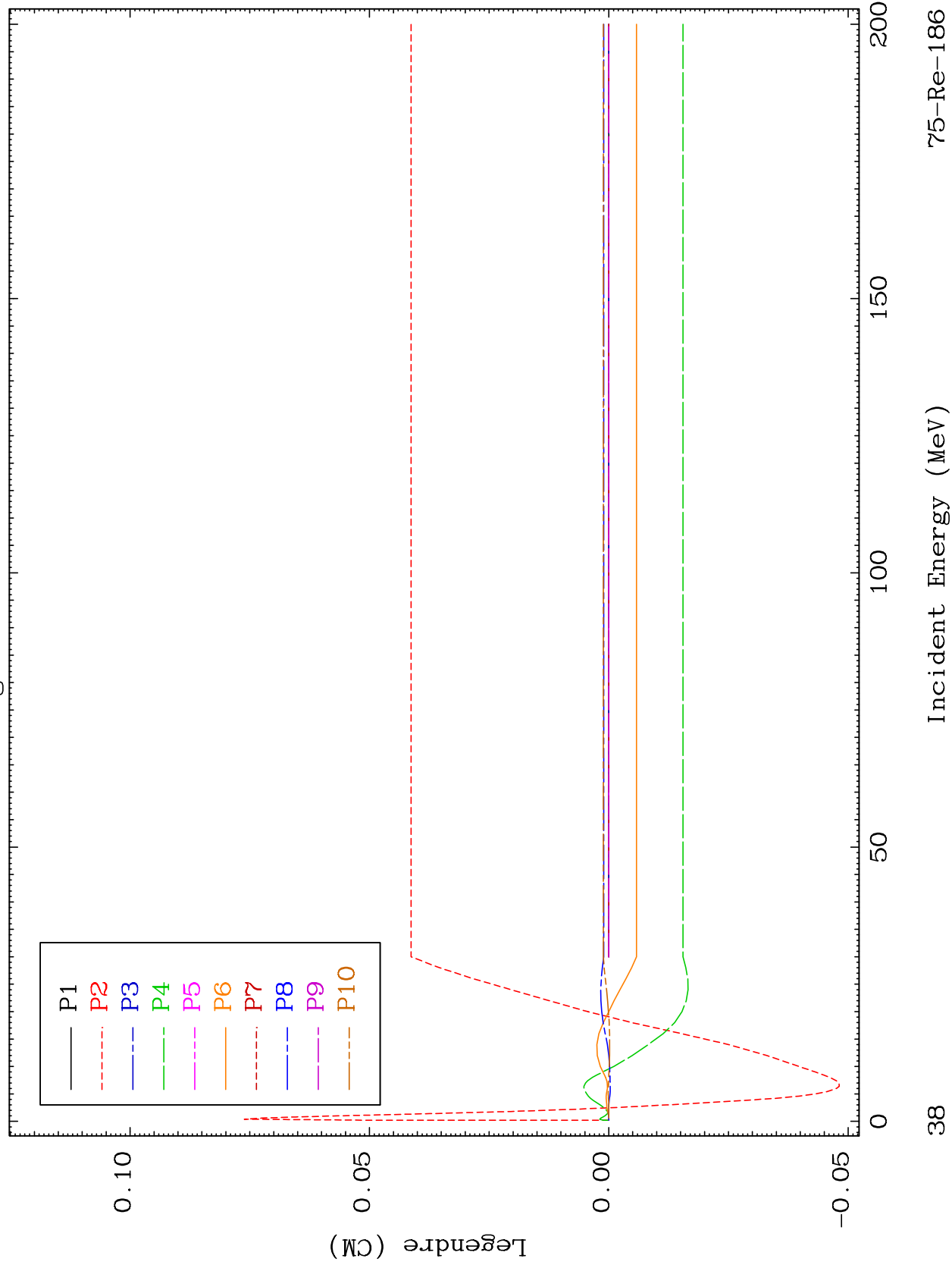


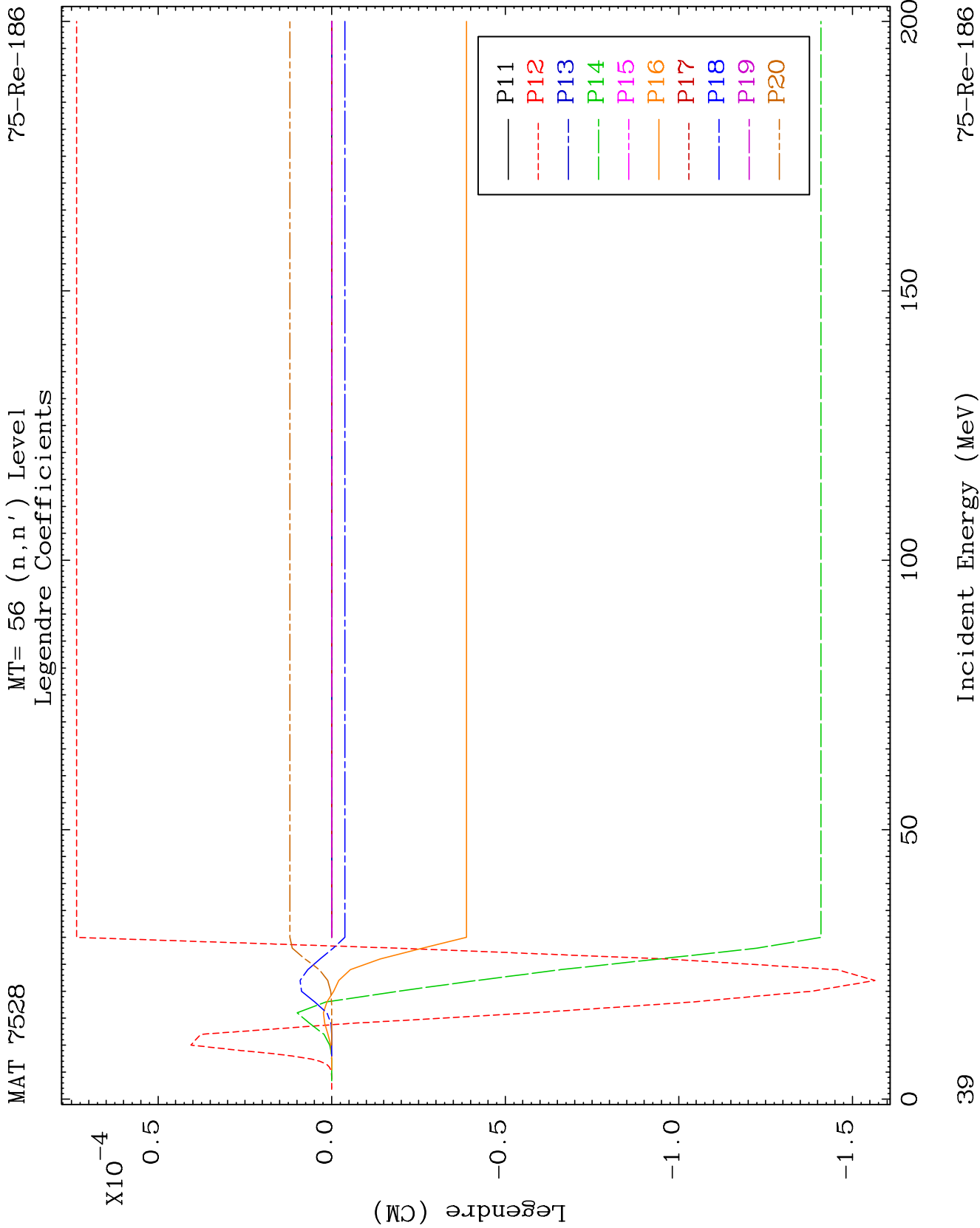


MAT 7528

MT= 56 (n, n') Level
Legendre Coefficients

75-Re-186

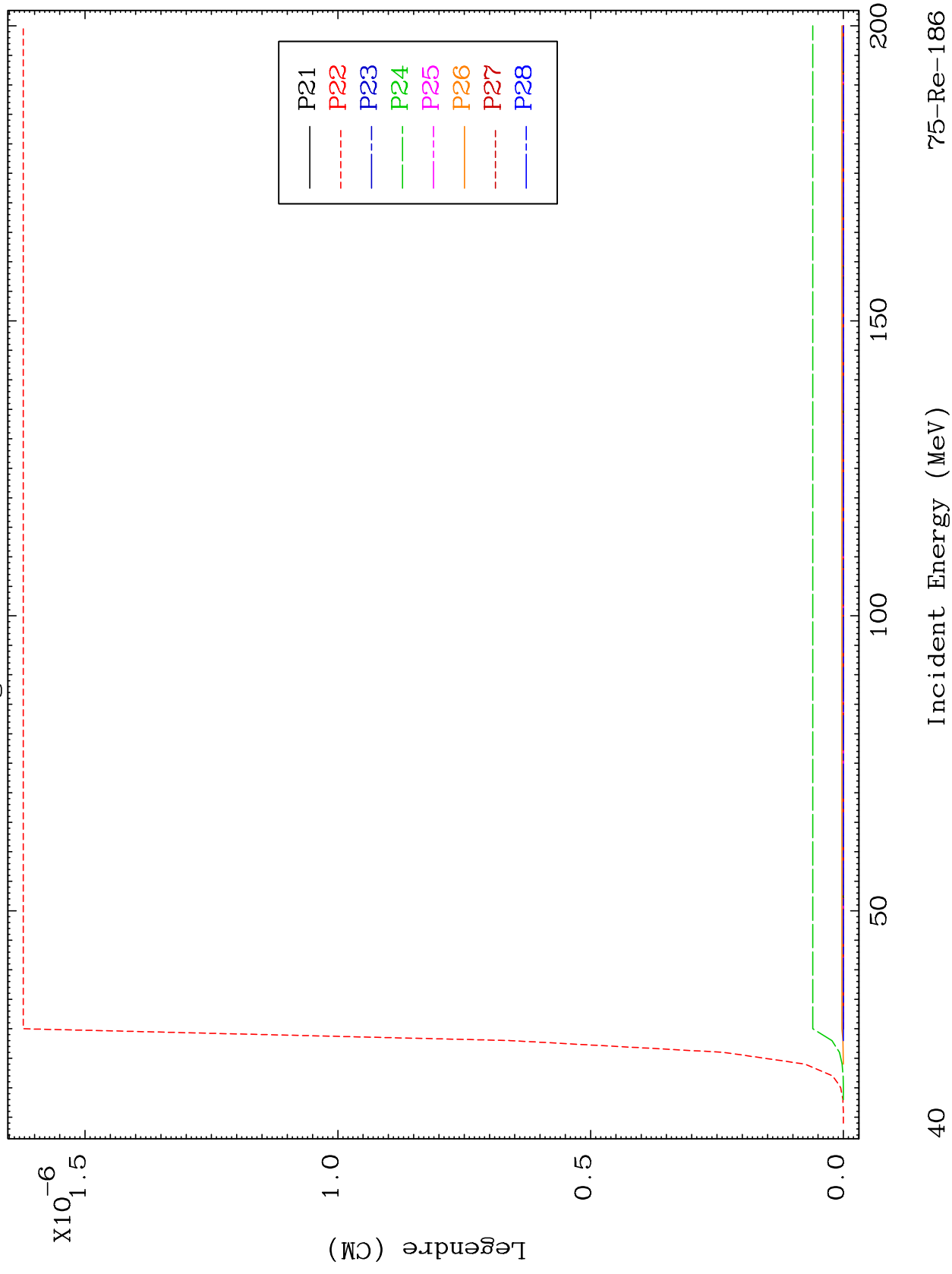




MAT 7528

MT= 56 (n, n') Level
Legendre Coefficients

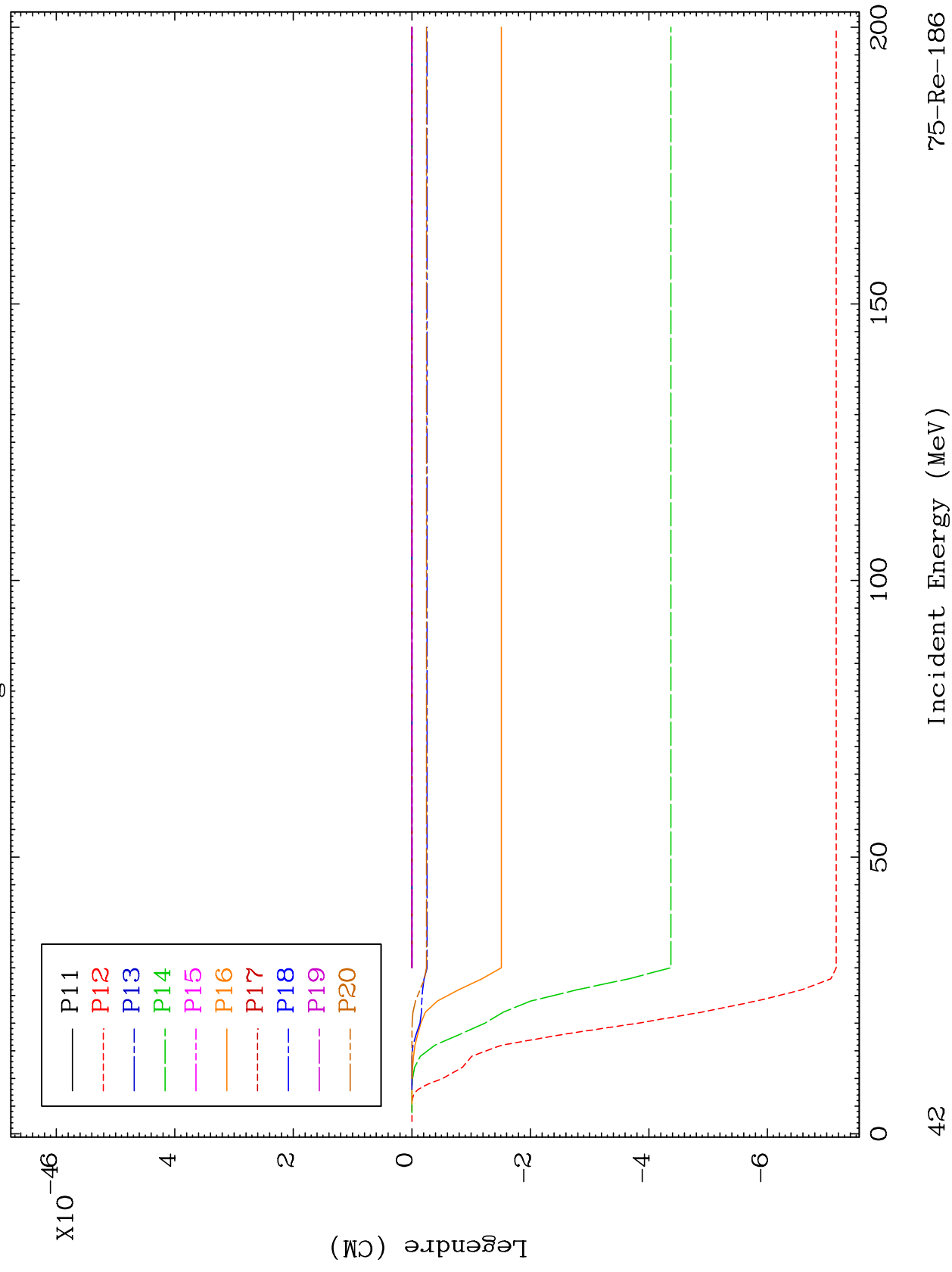
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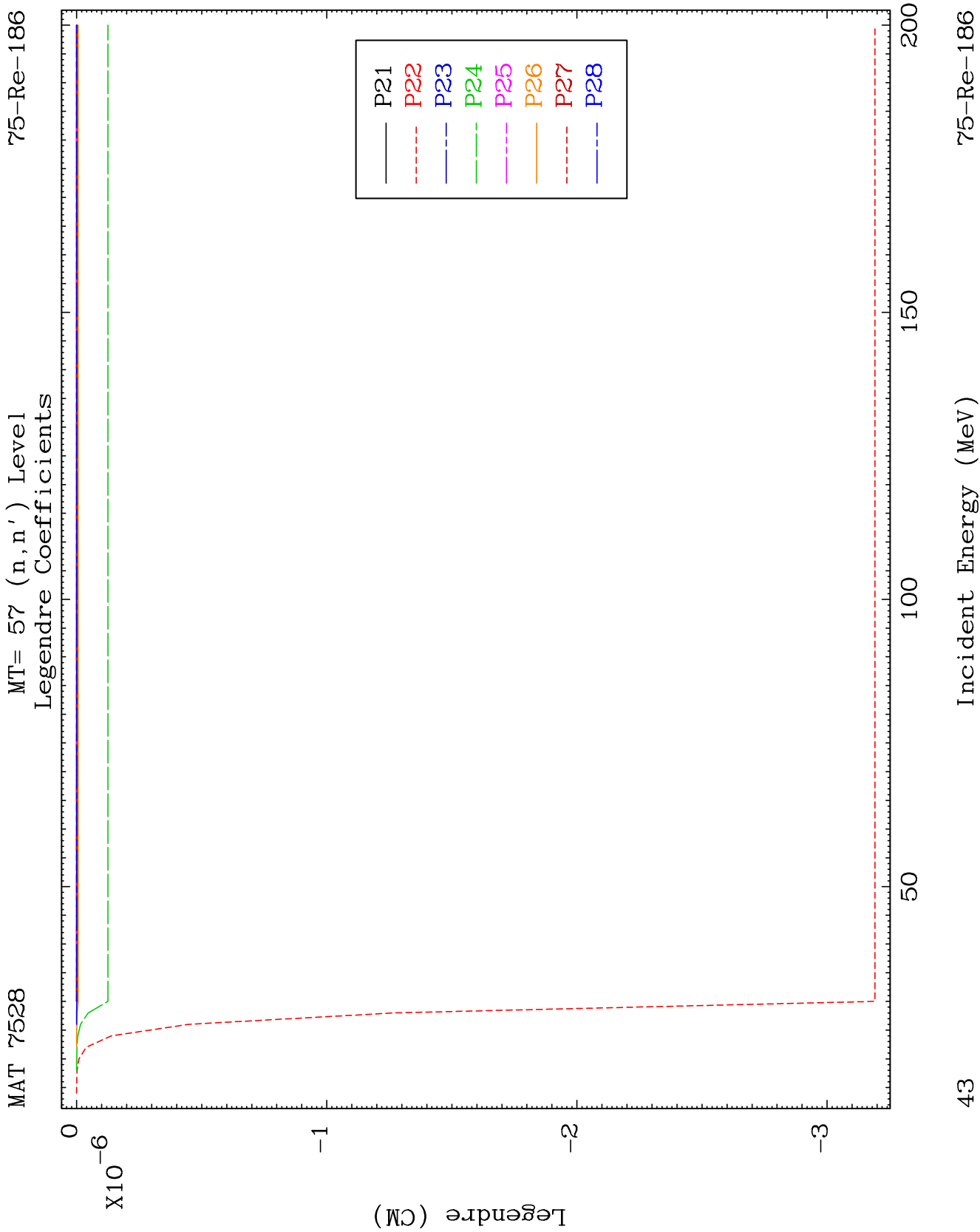


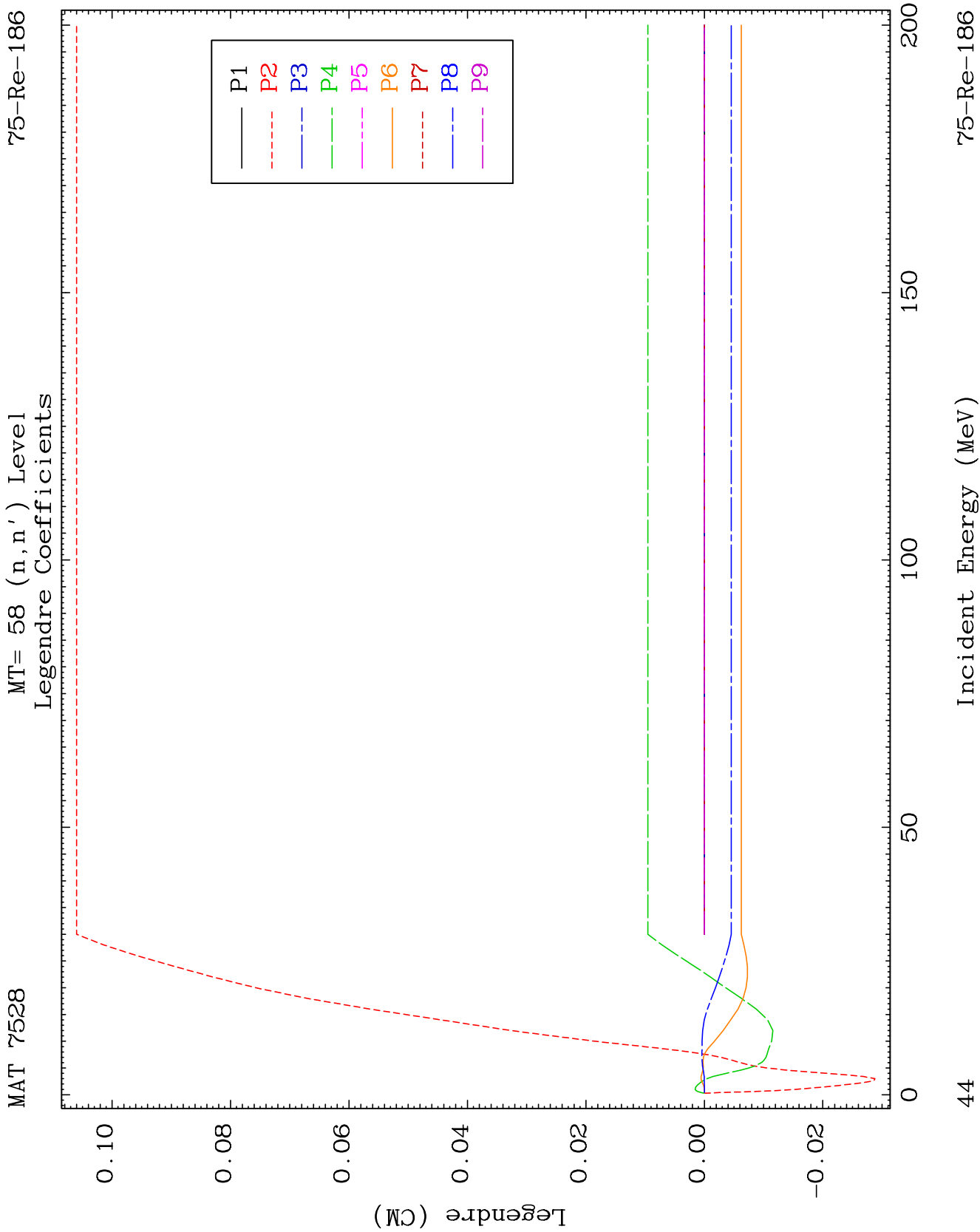
MAT 7528

MT= 57 (n, n') Level
Legendre Coefficients

75-Re-186



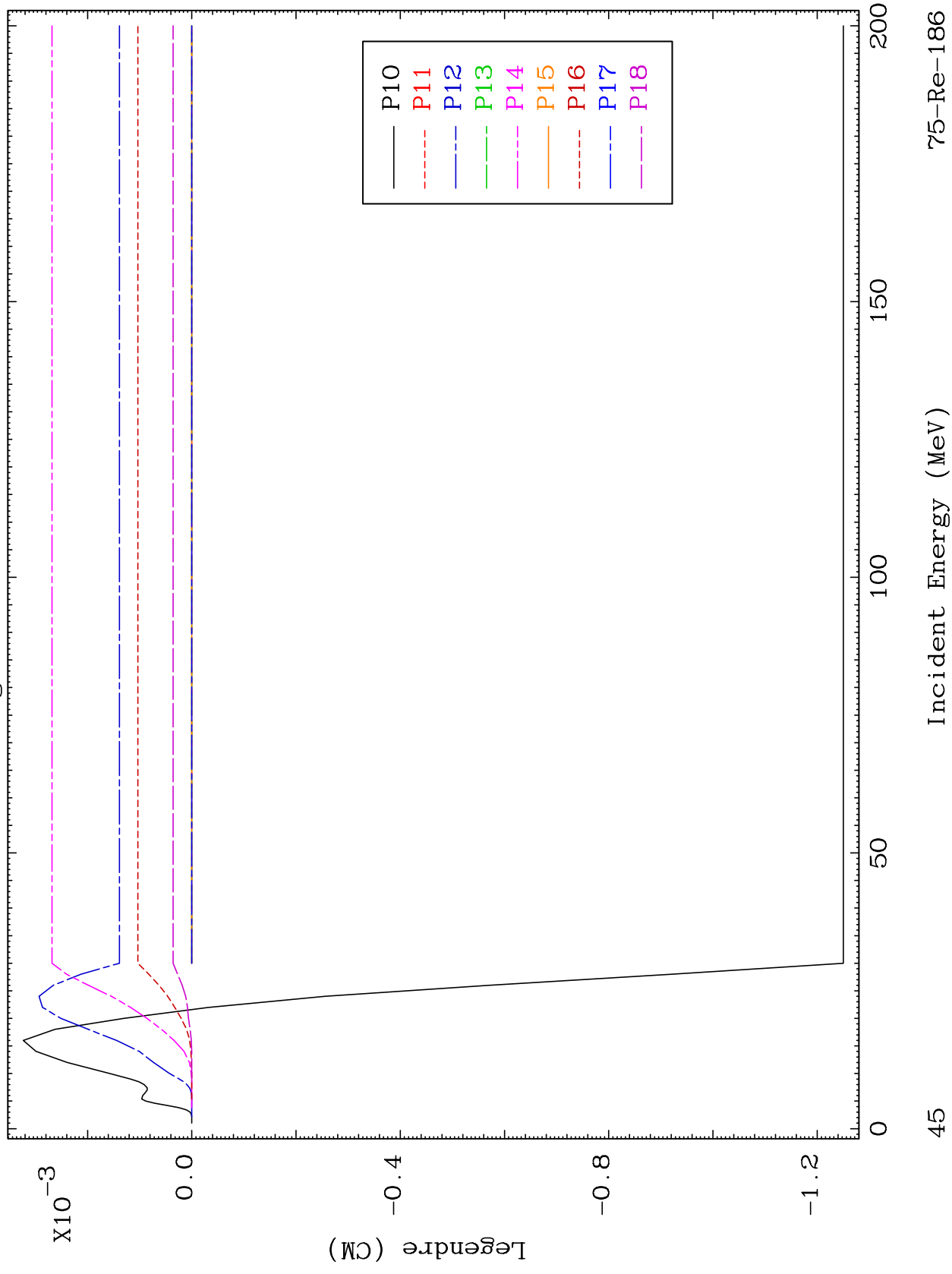


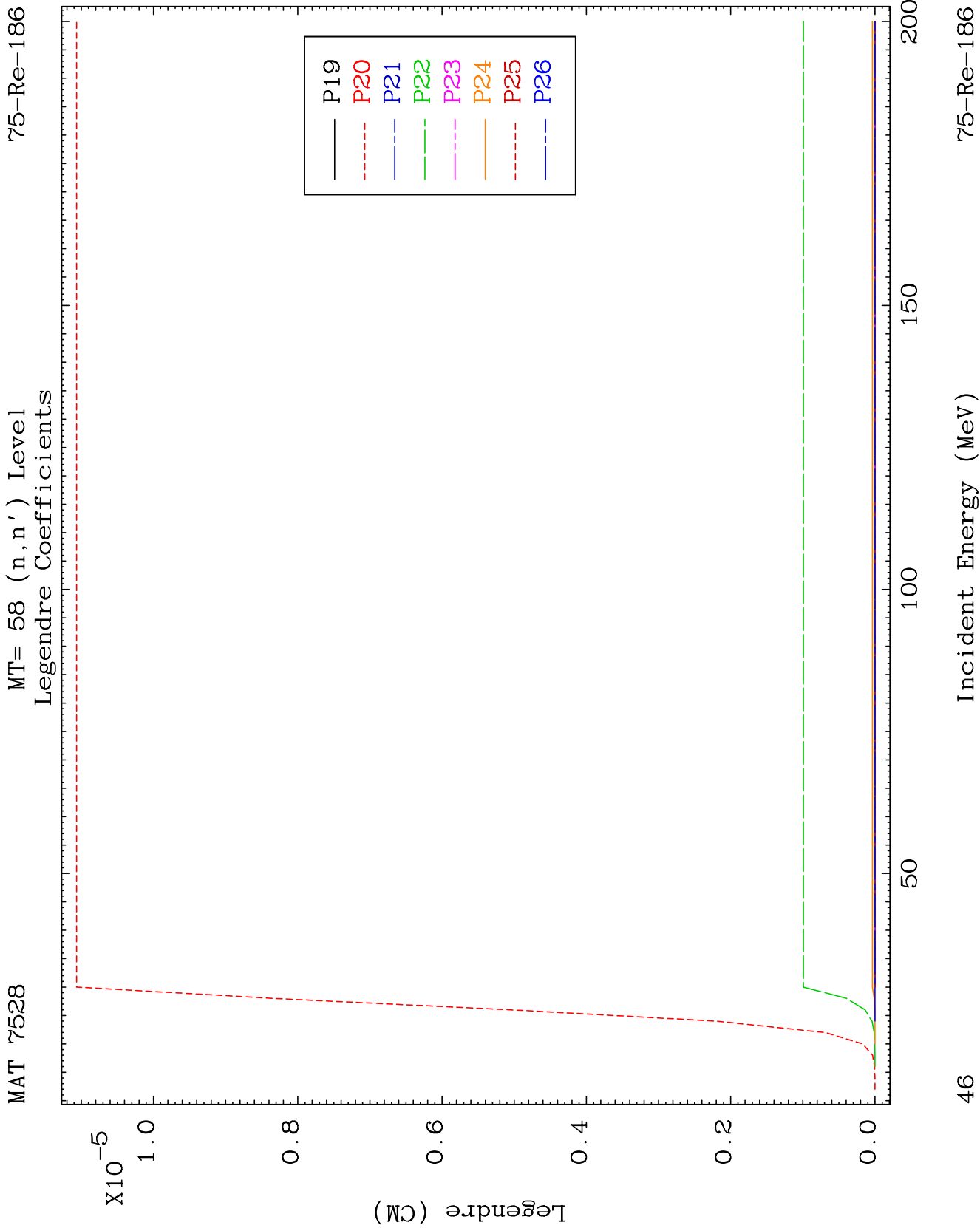


MAT 7528

MT= 58 (n, n') Level
Legendre Coefficients

75-Re-186

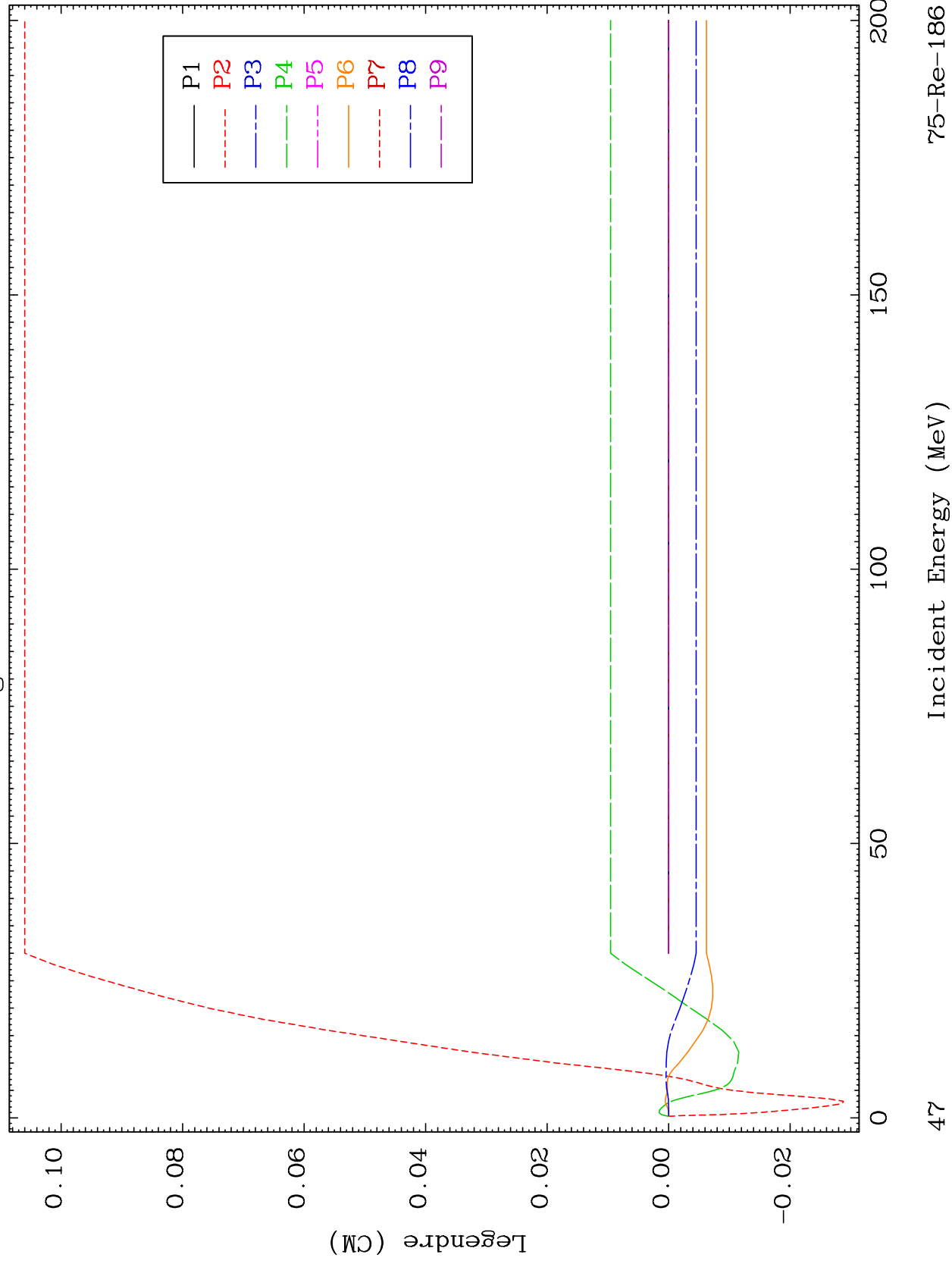


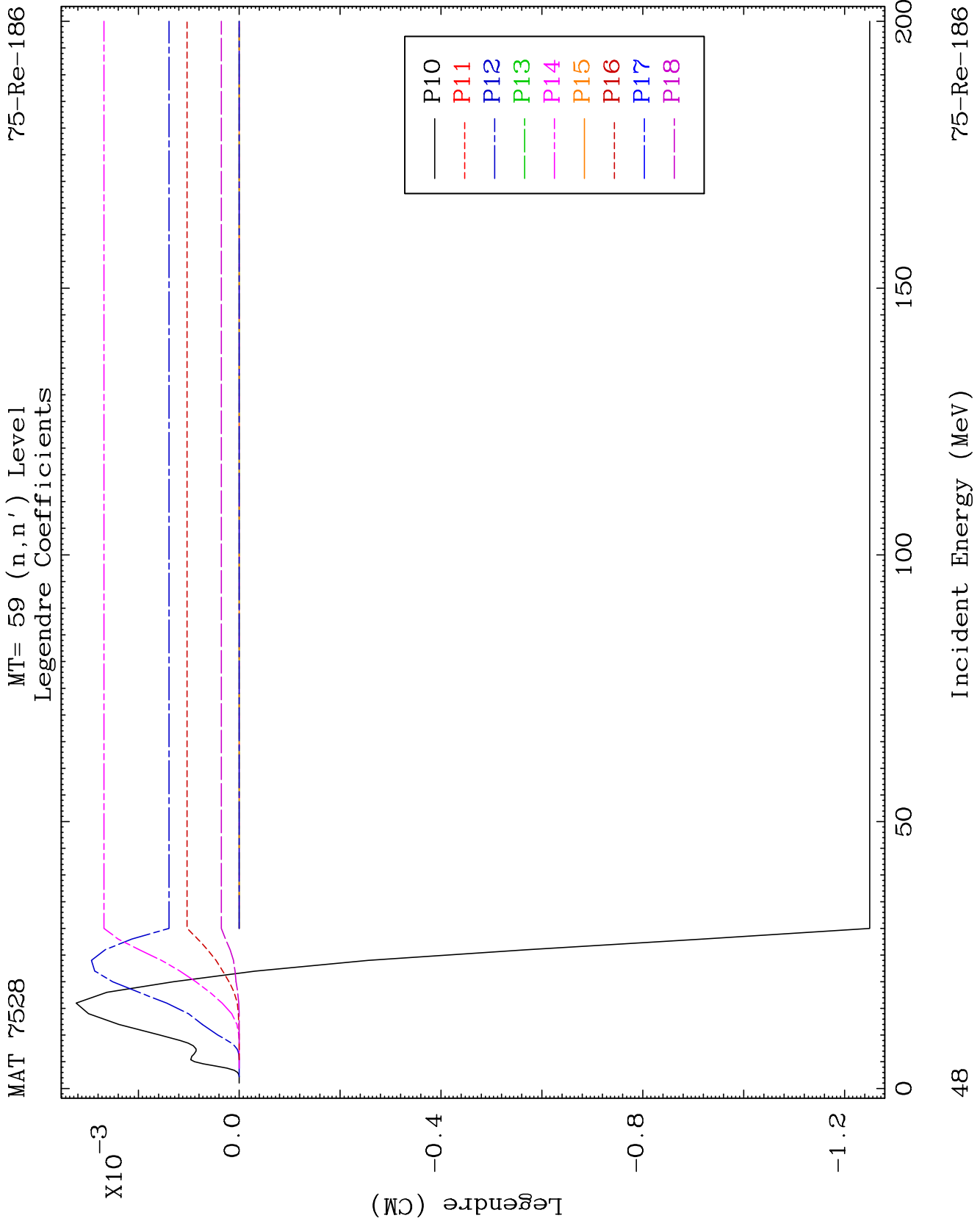


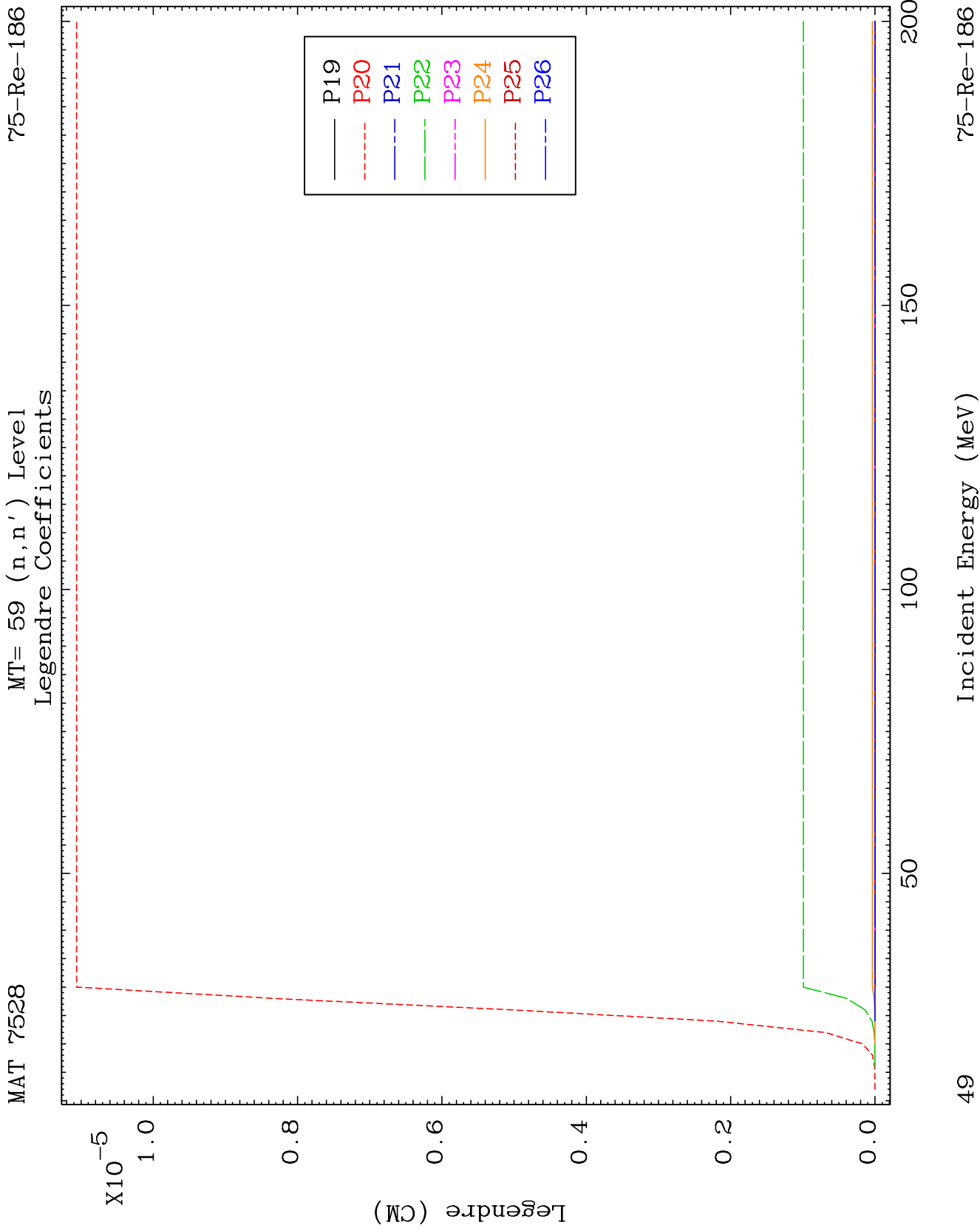
MAT 7528

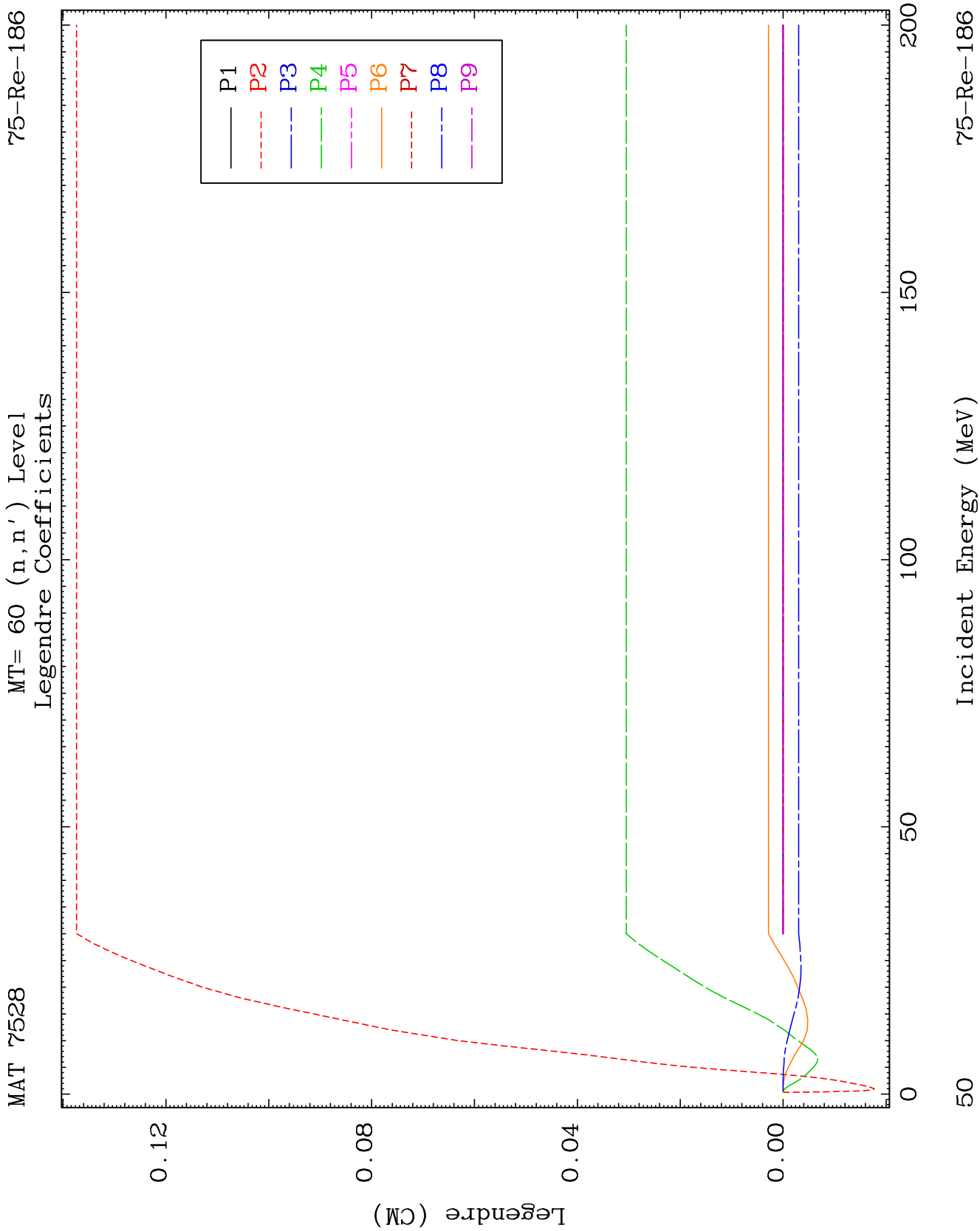
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Legendre Coefficients

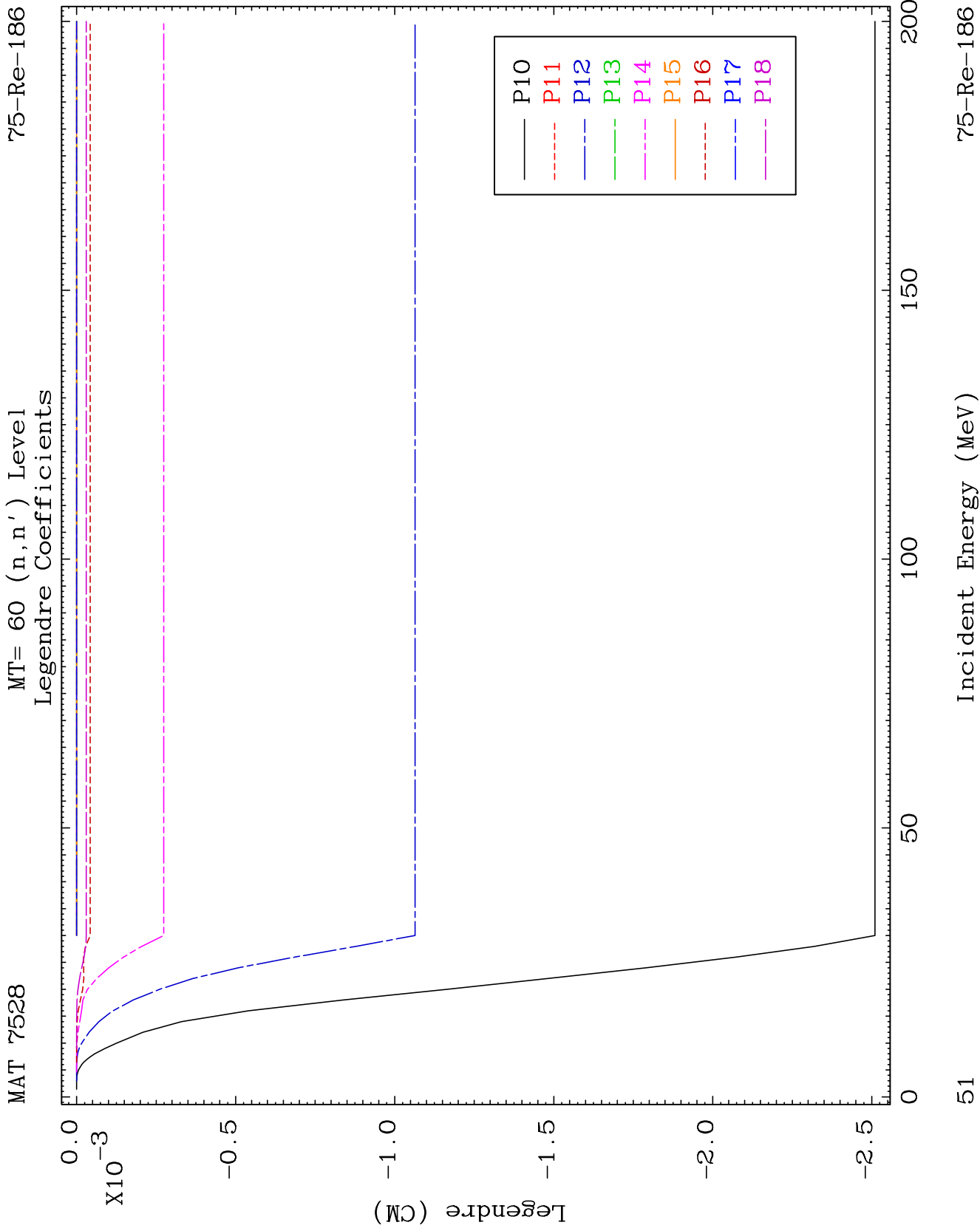
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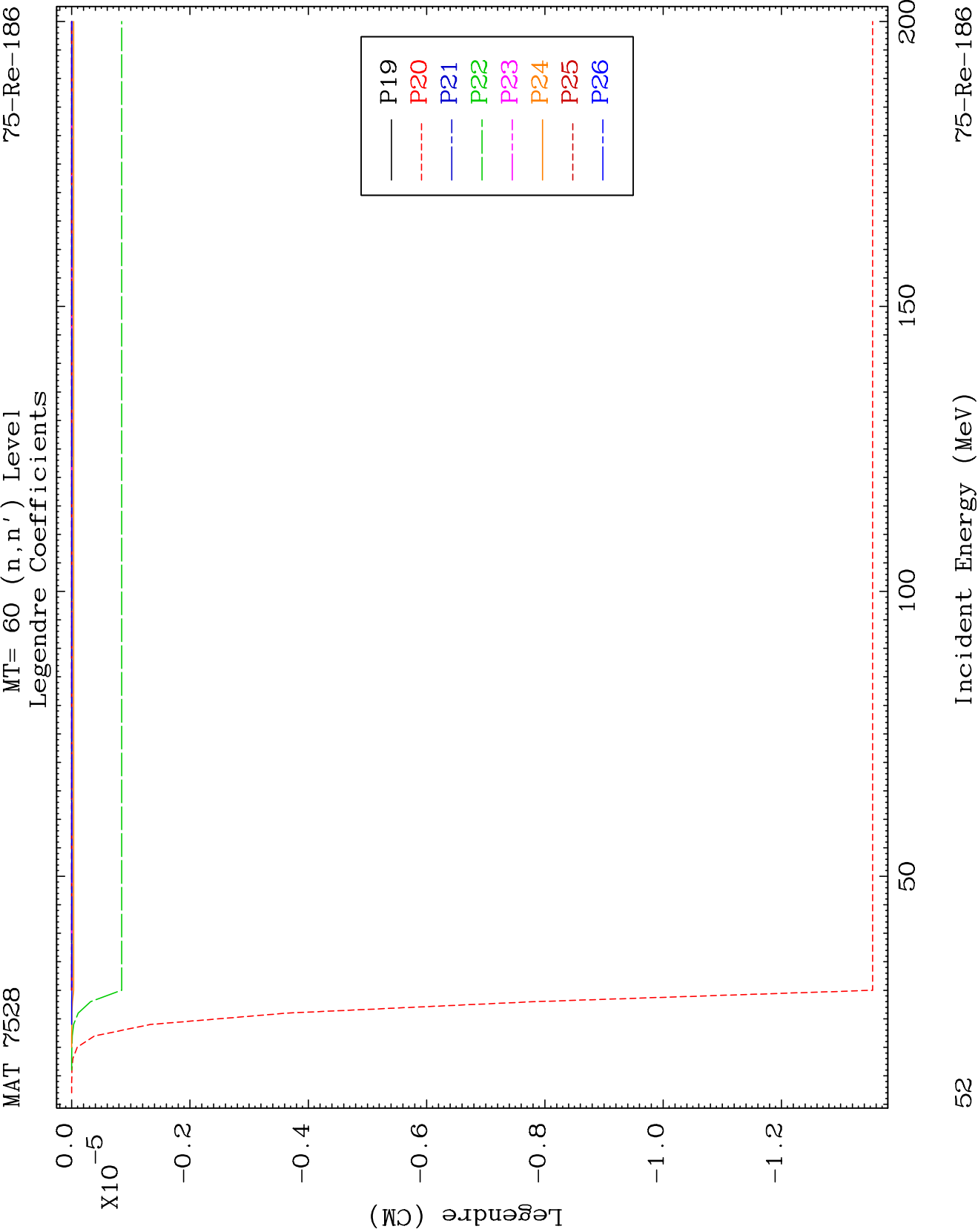


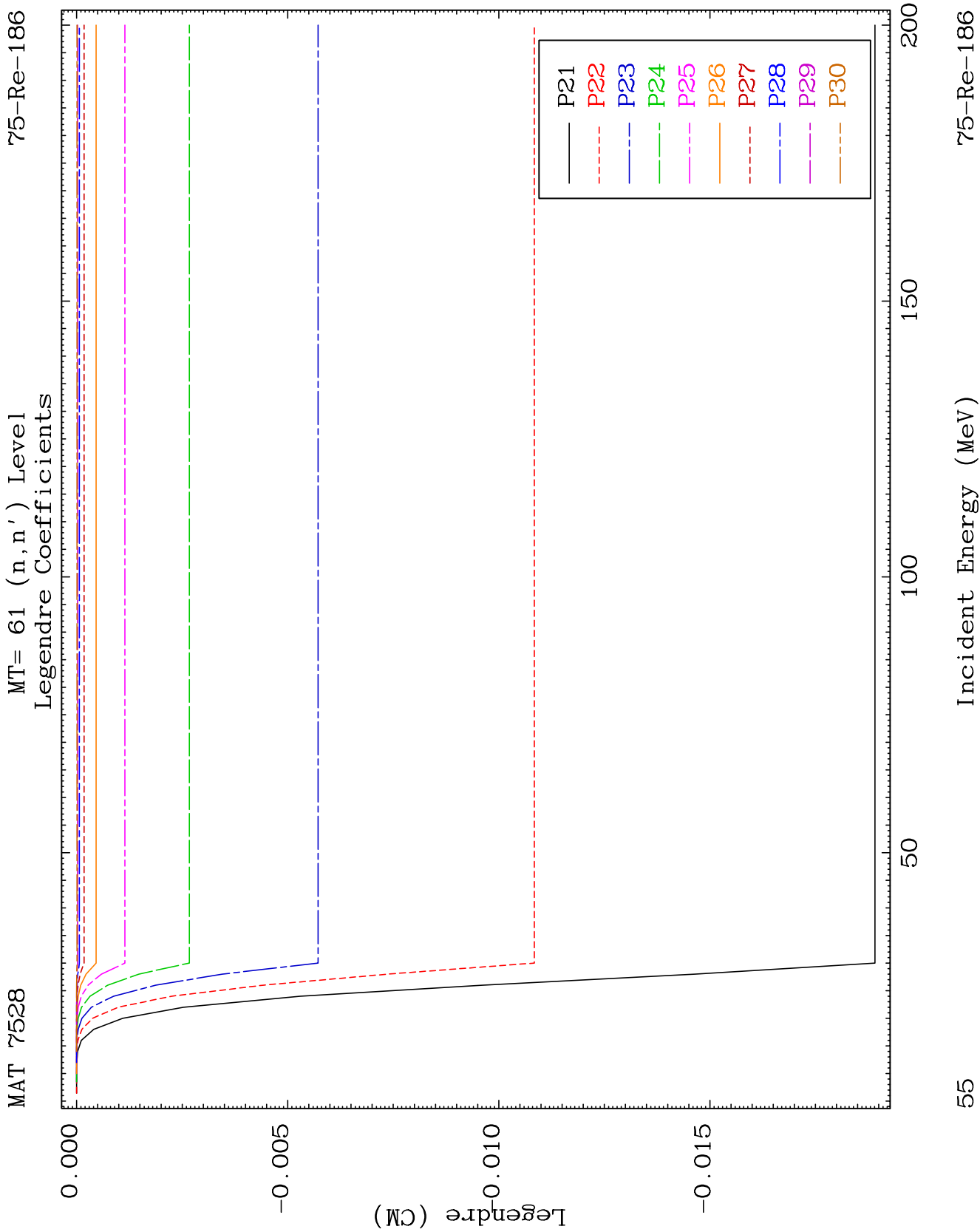


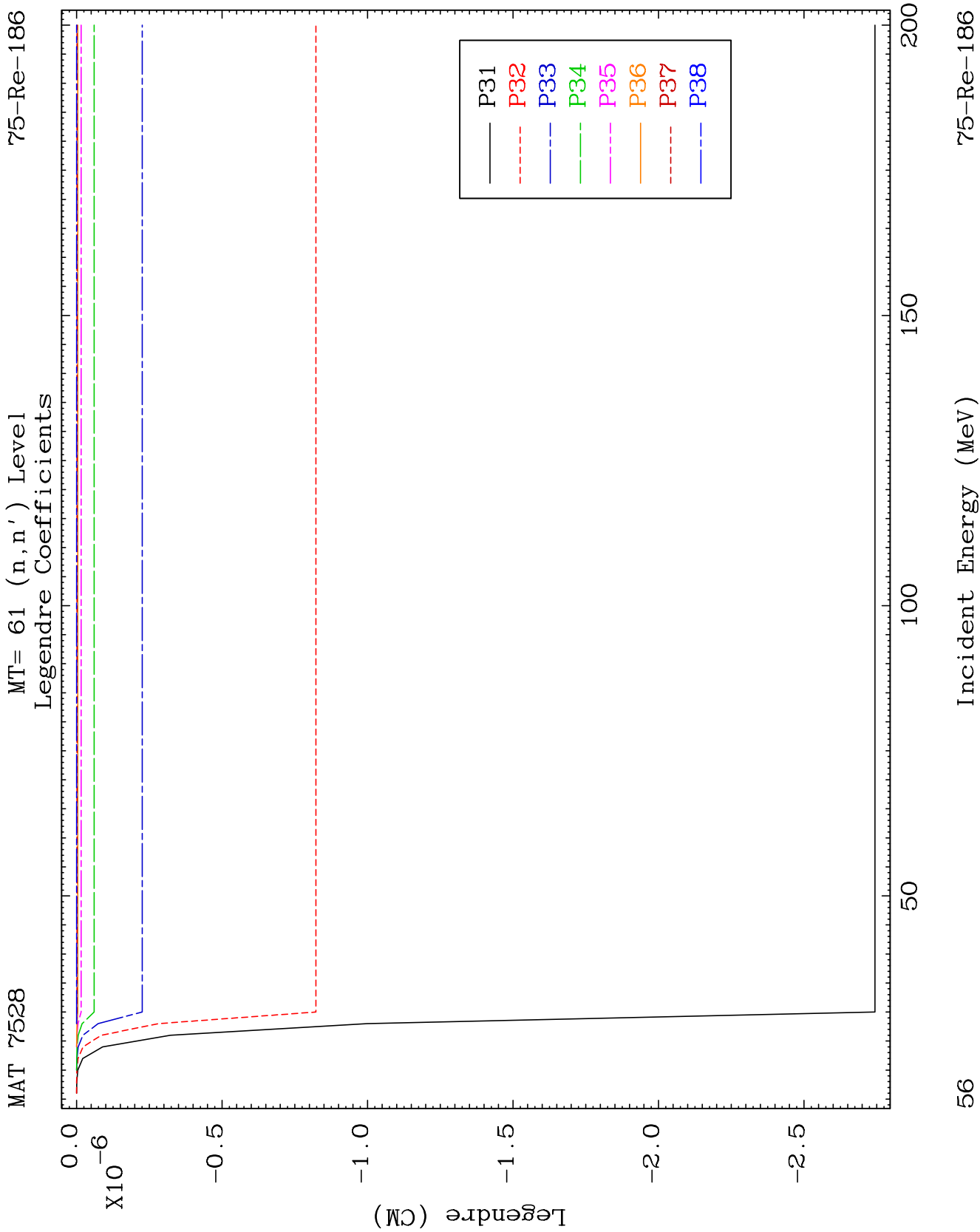


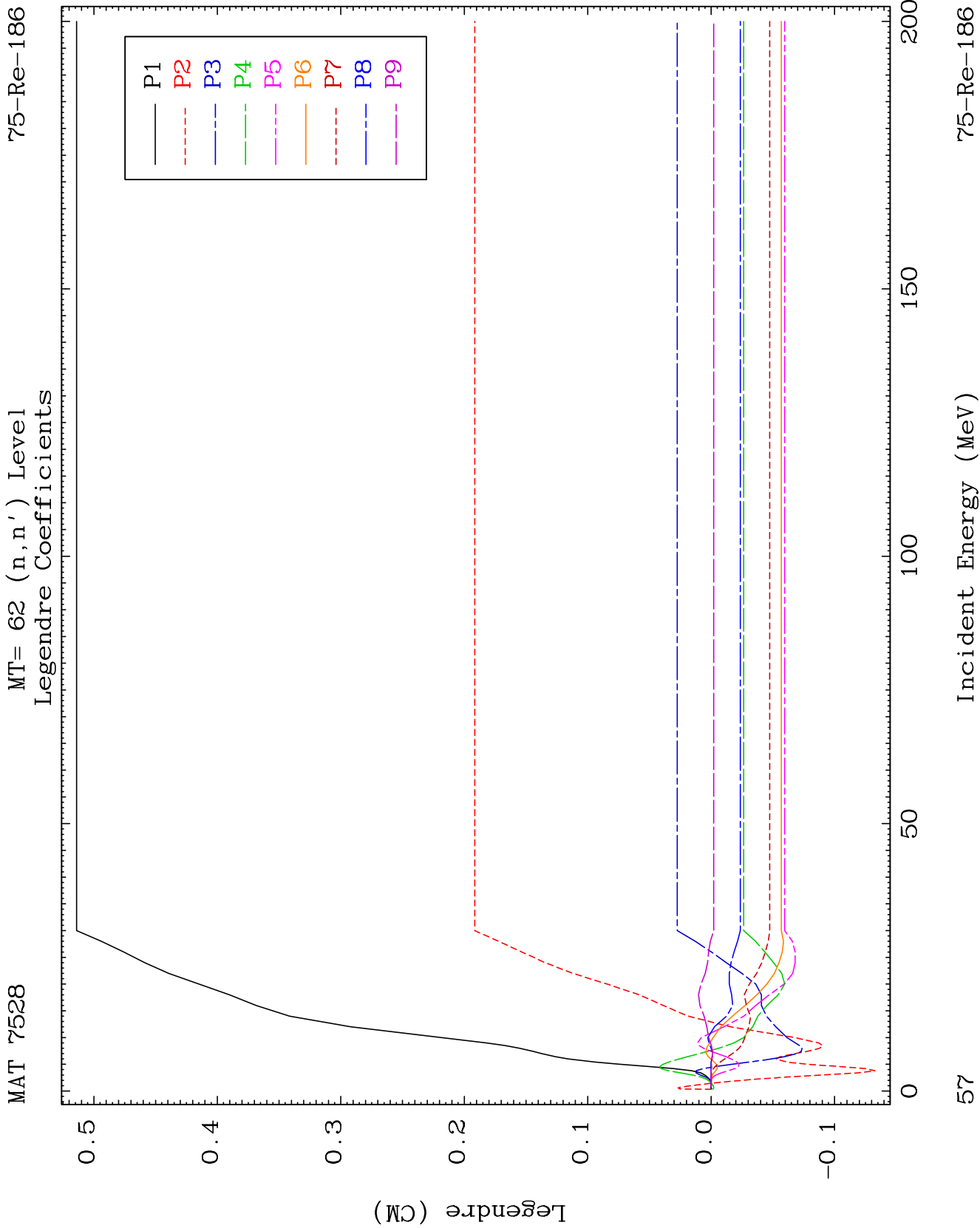


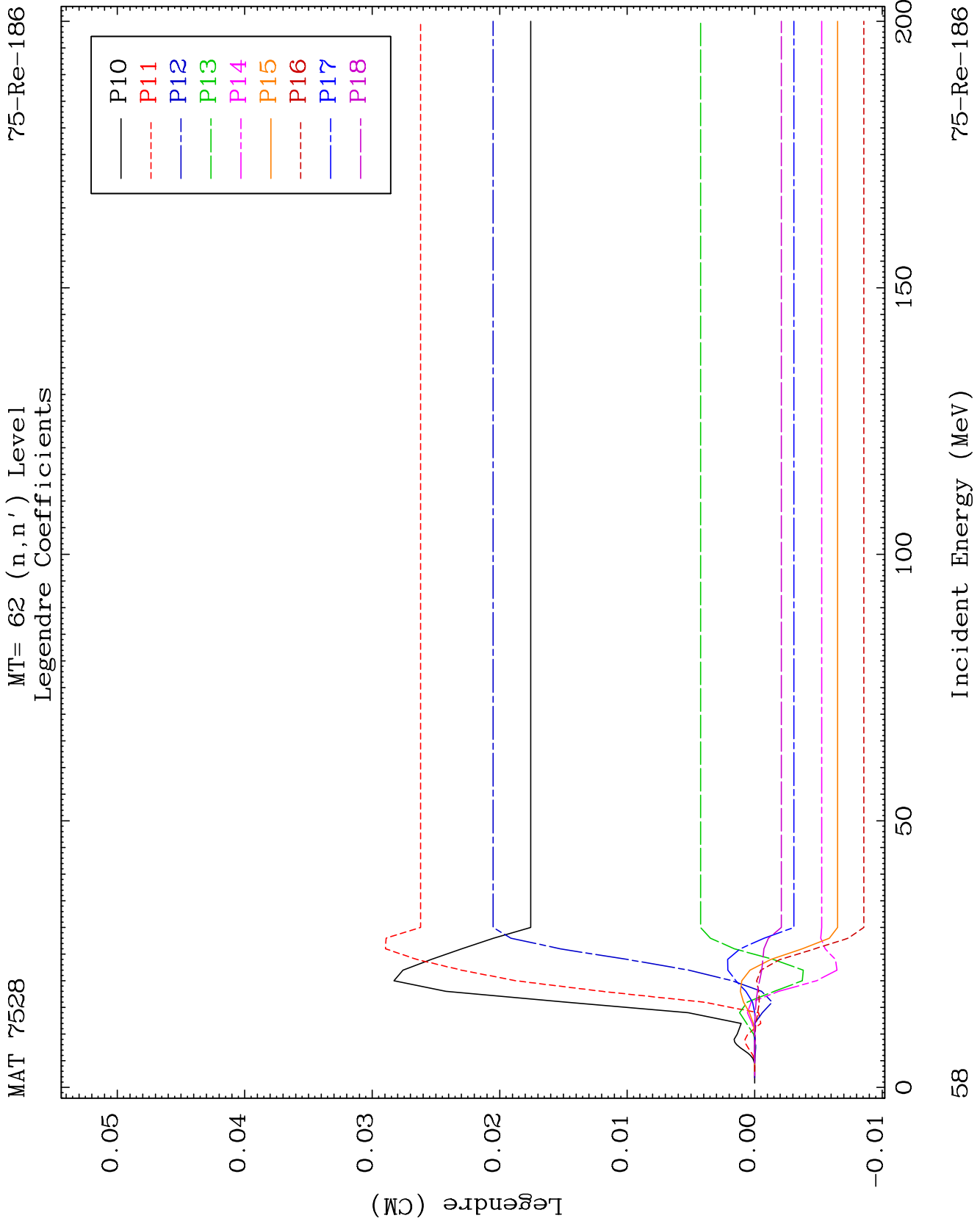


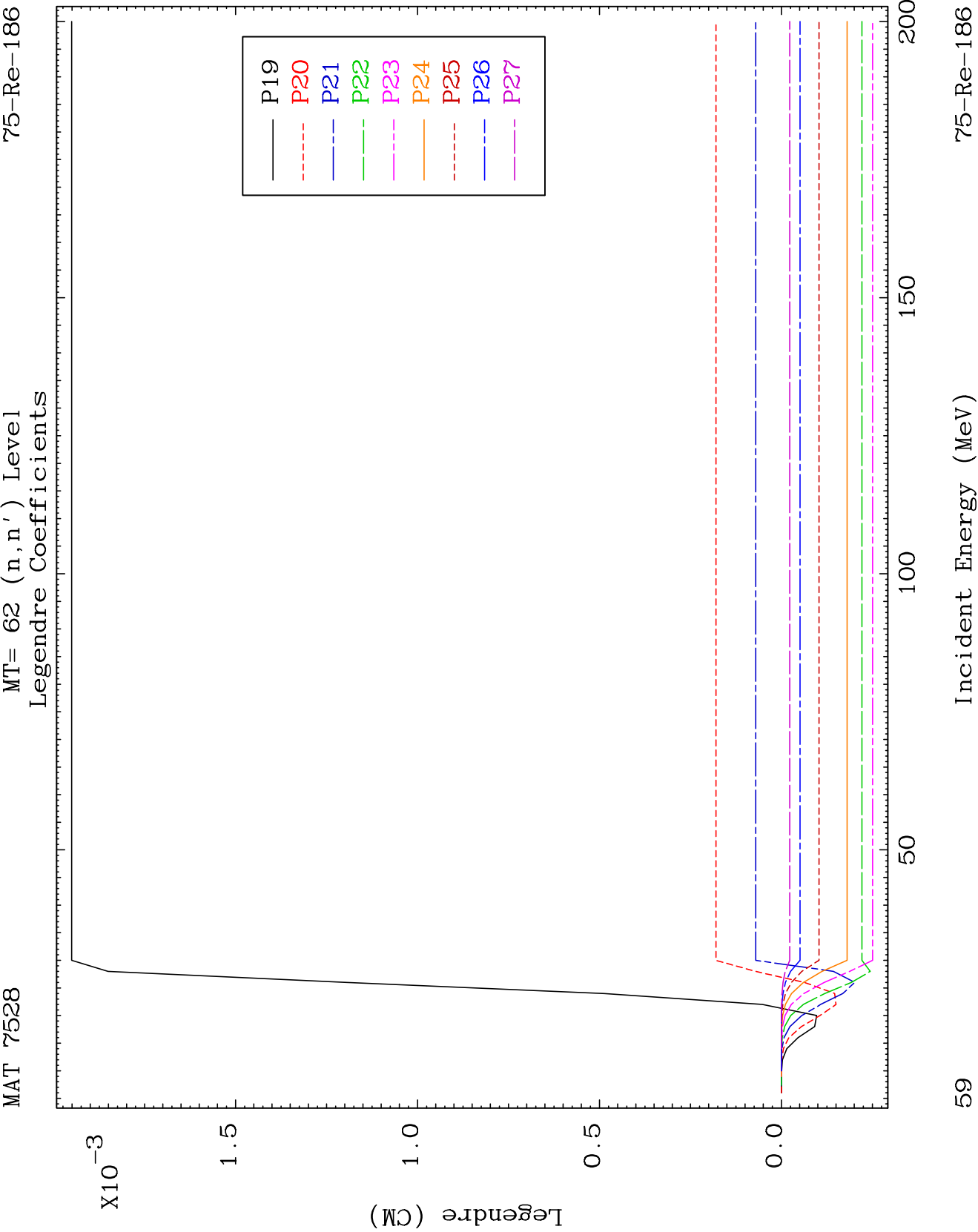


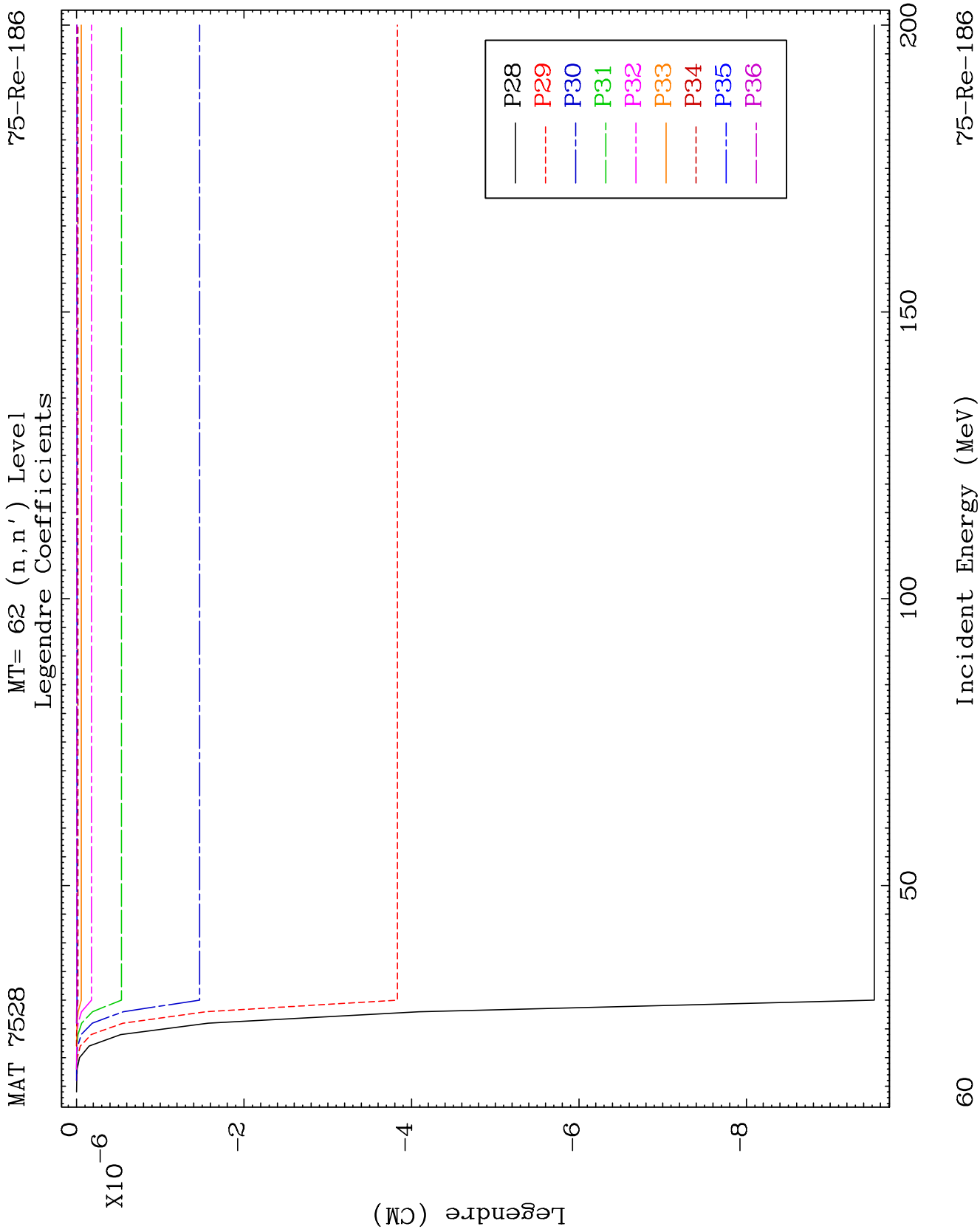


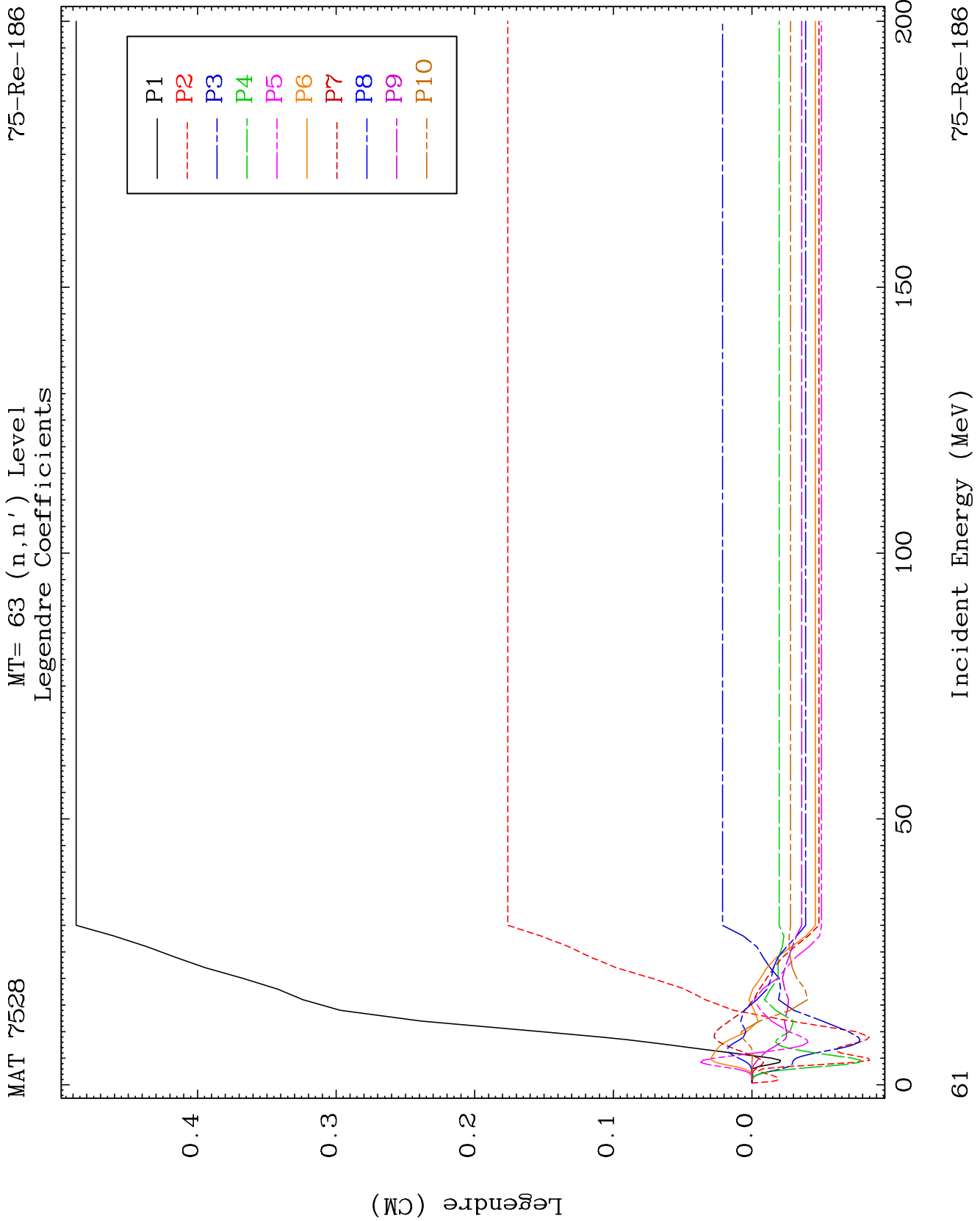


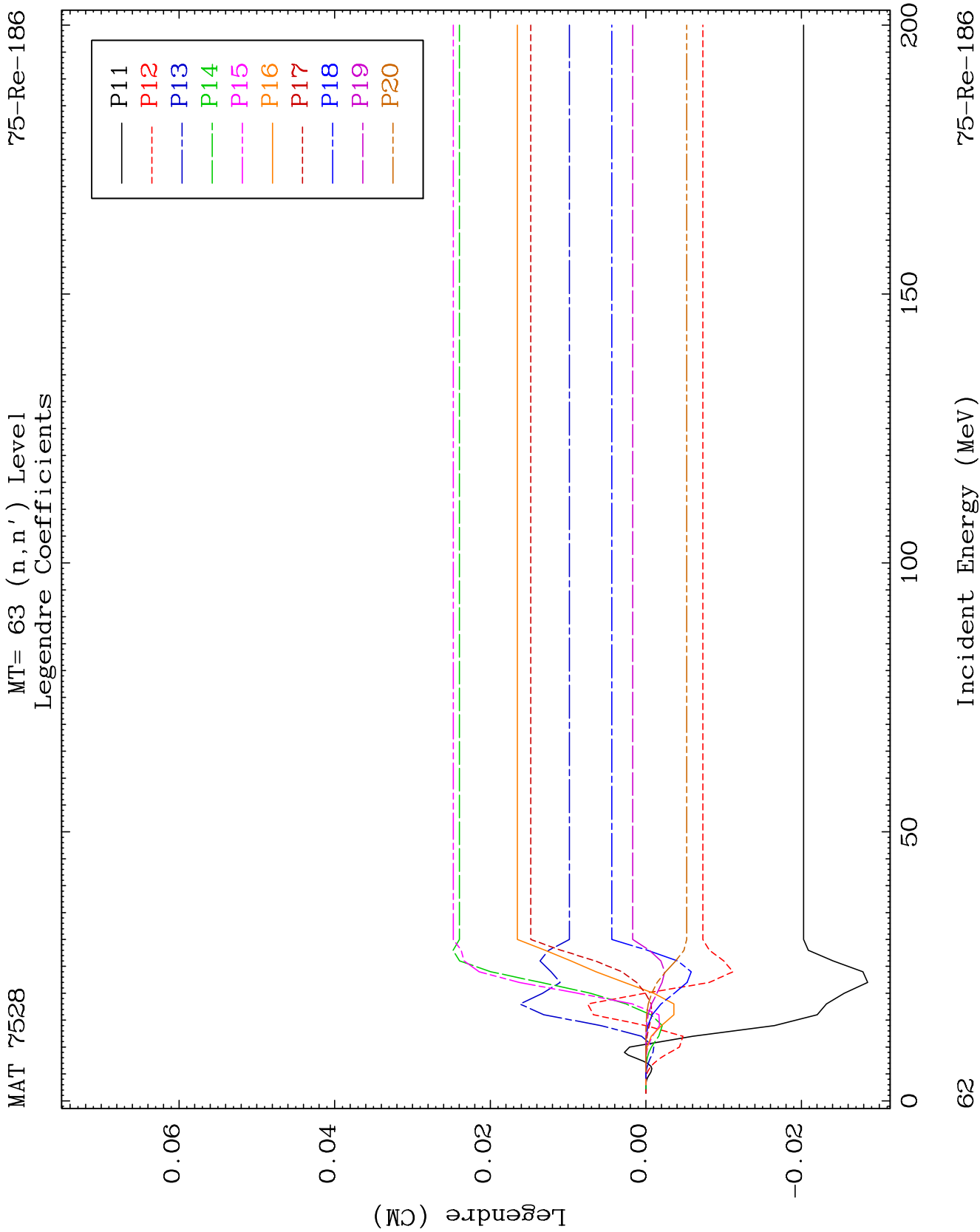


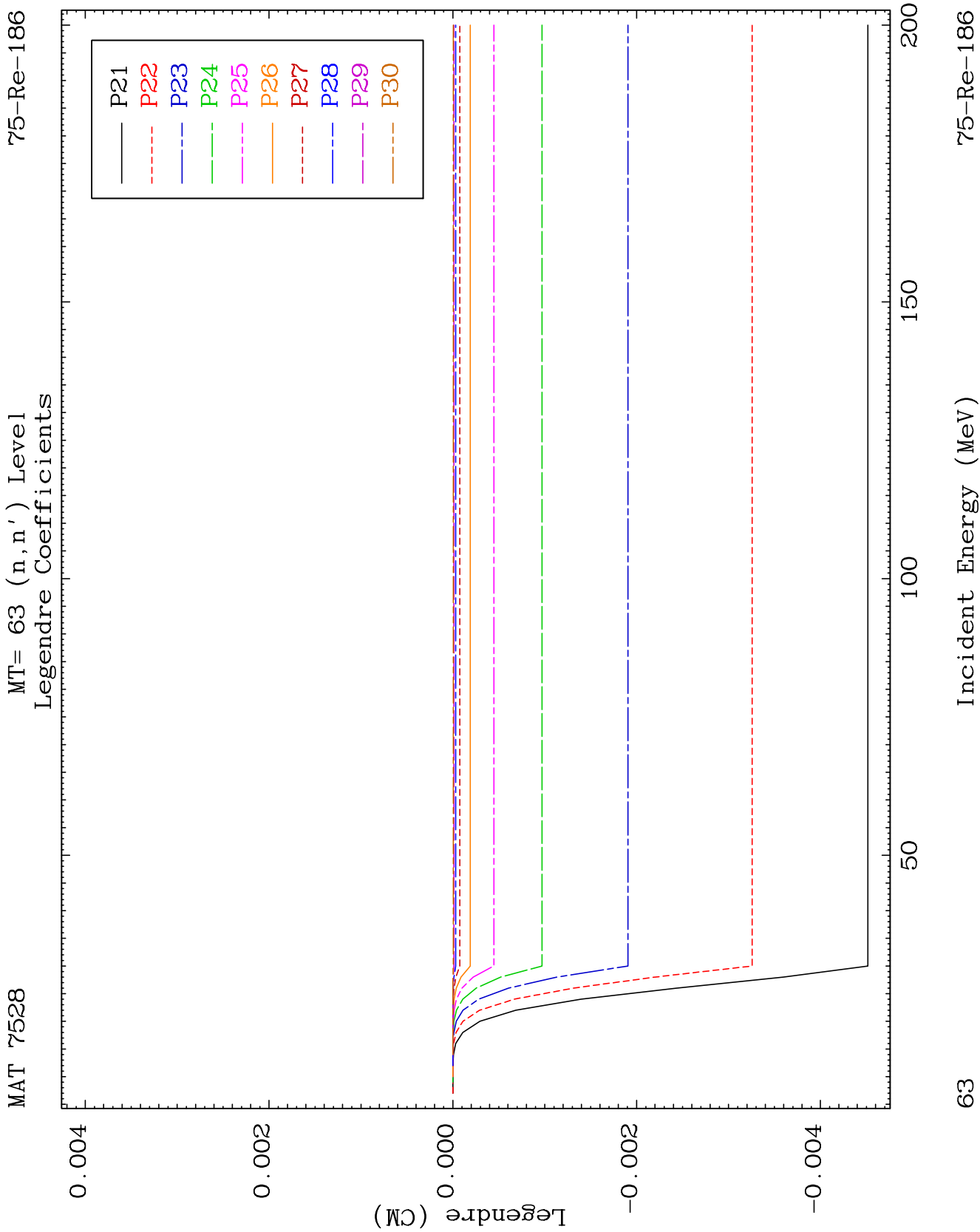


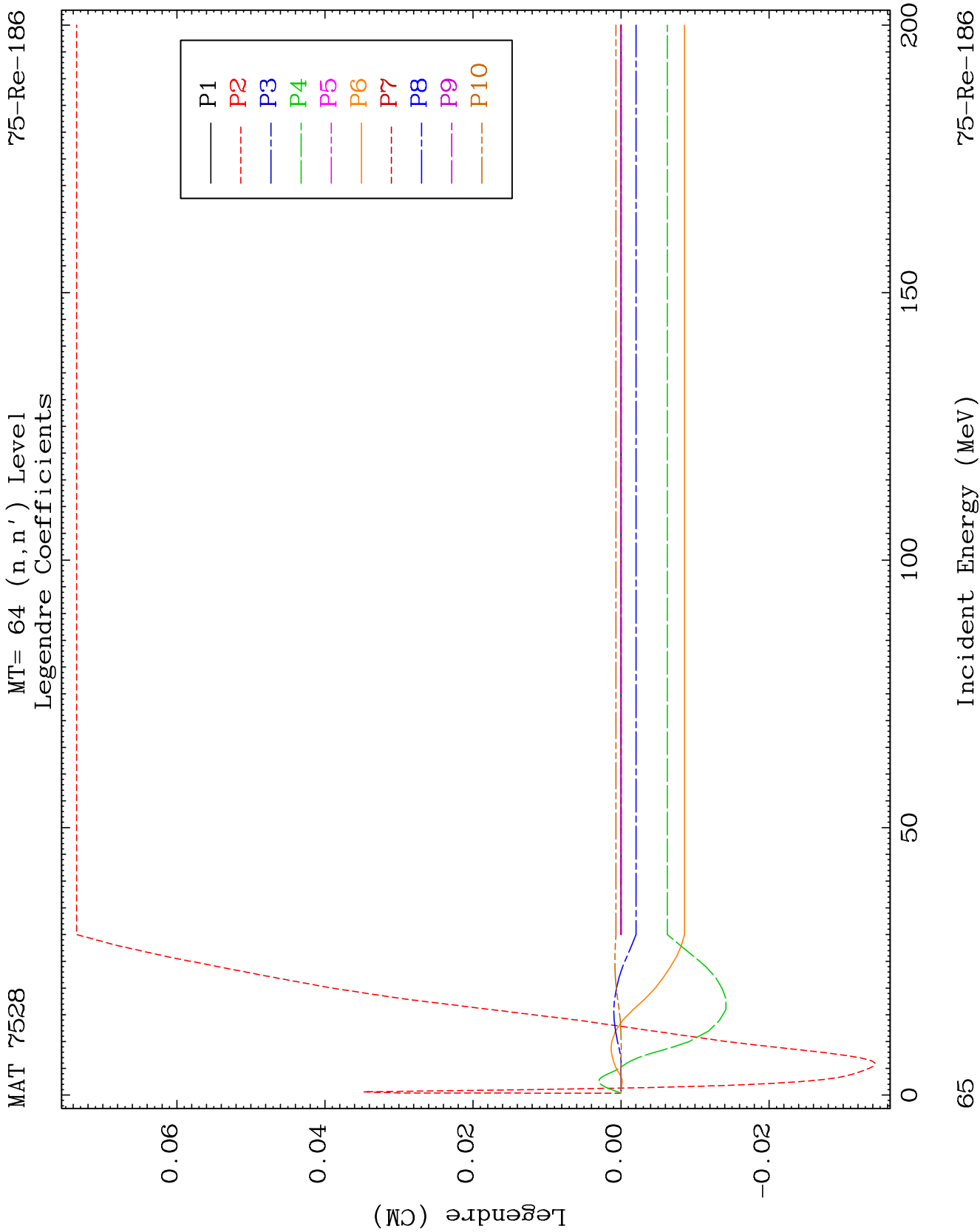


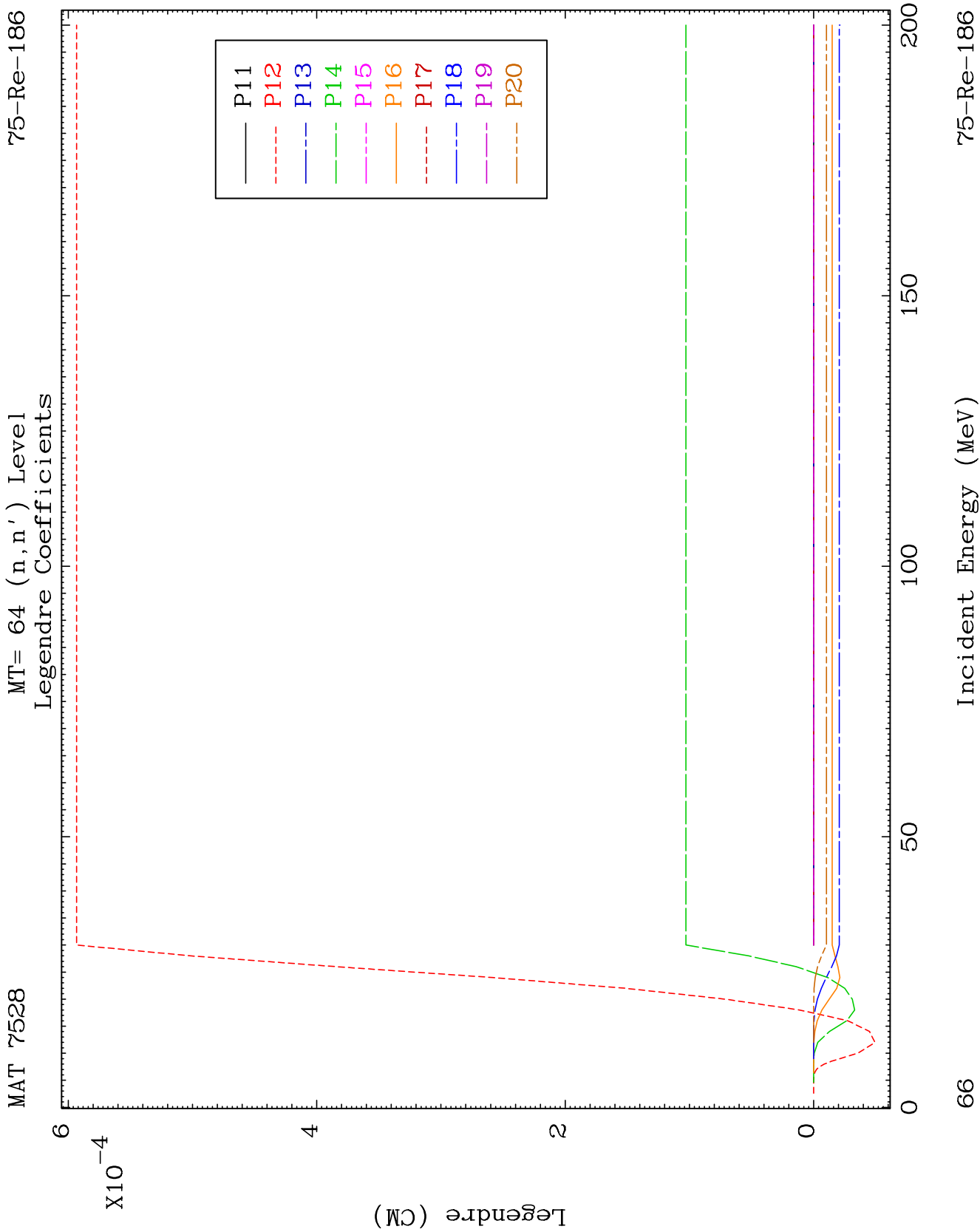


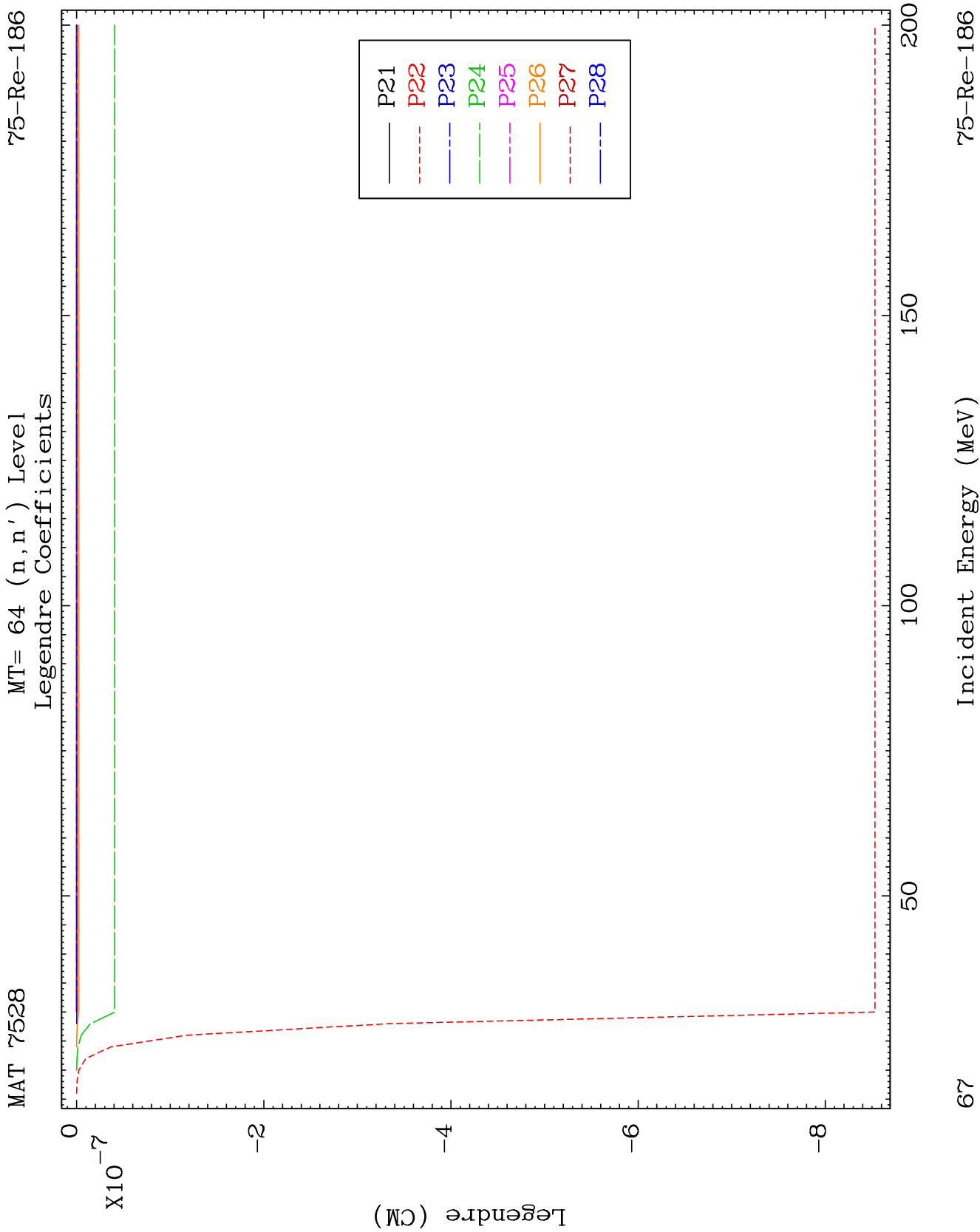








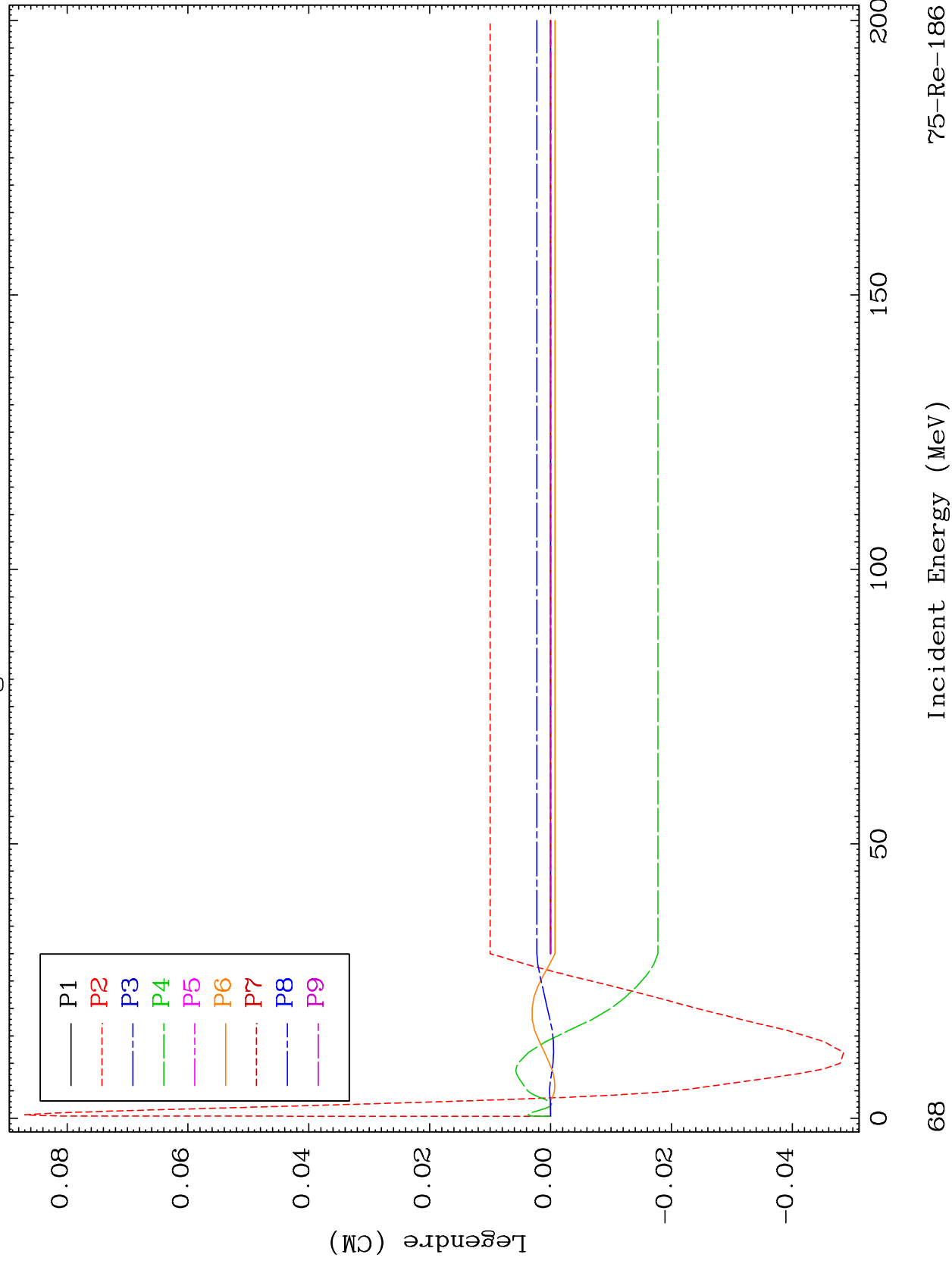


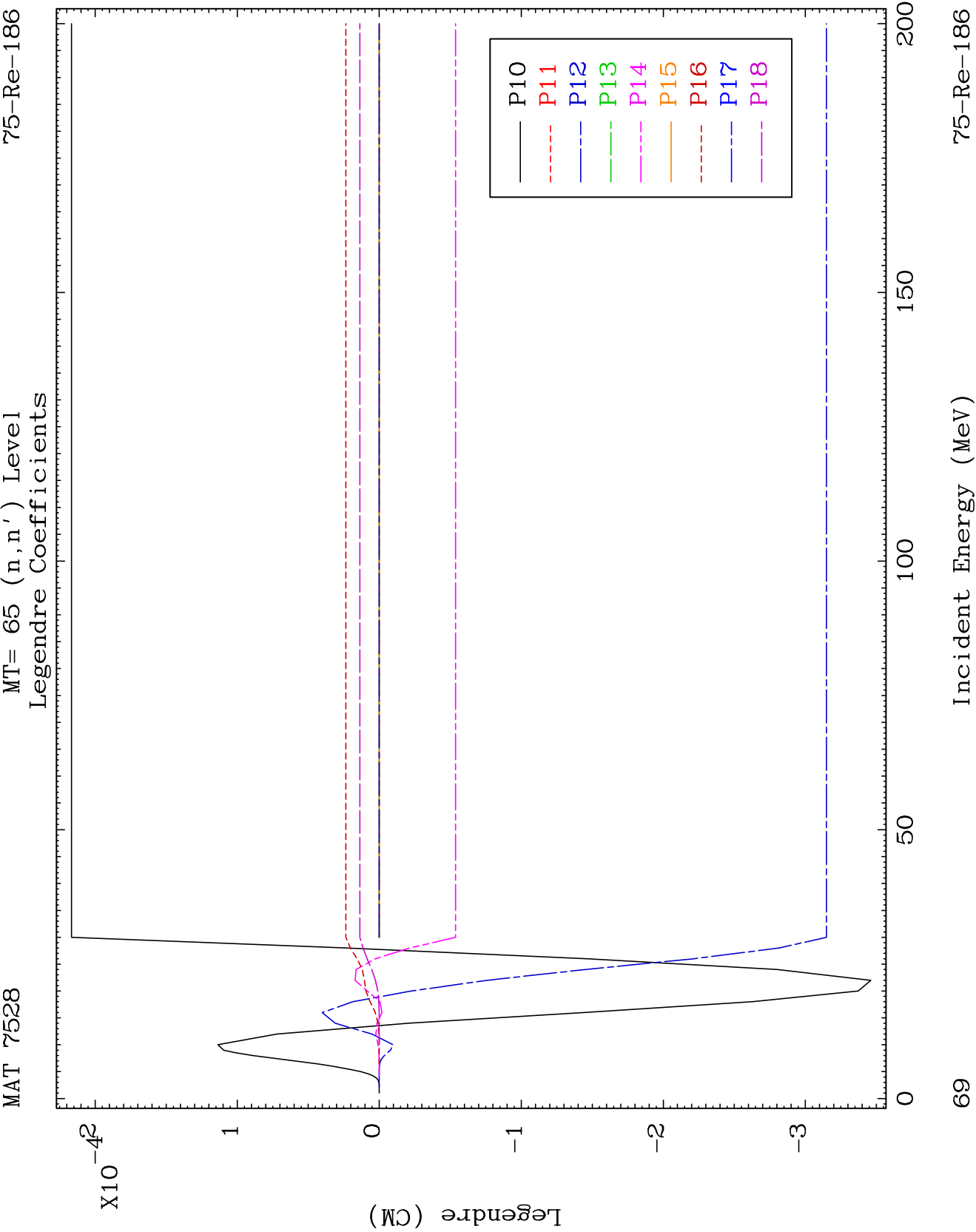


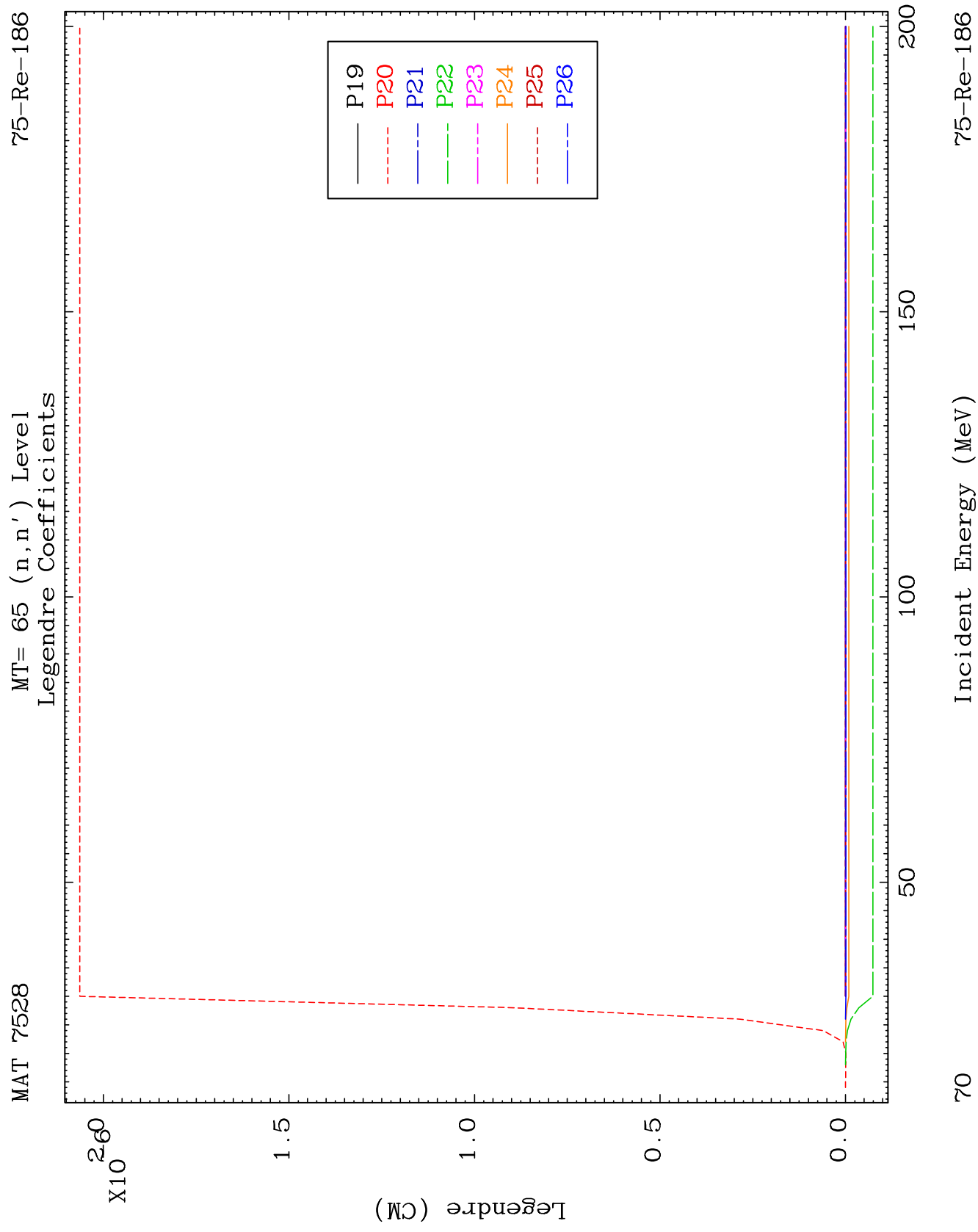
MAT 7528

MT= 65 (n, n') Level
Legendre Coefficients

75-Re-186



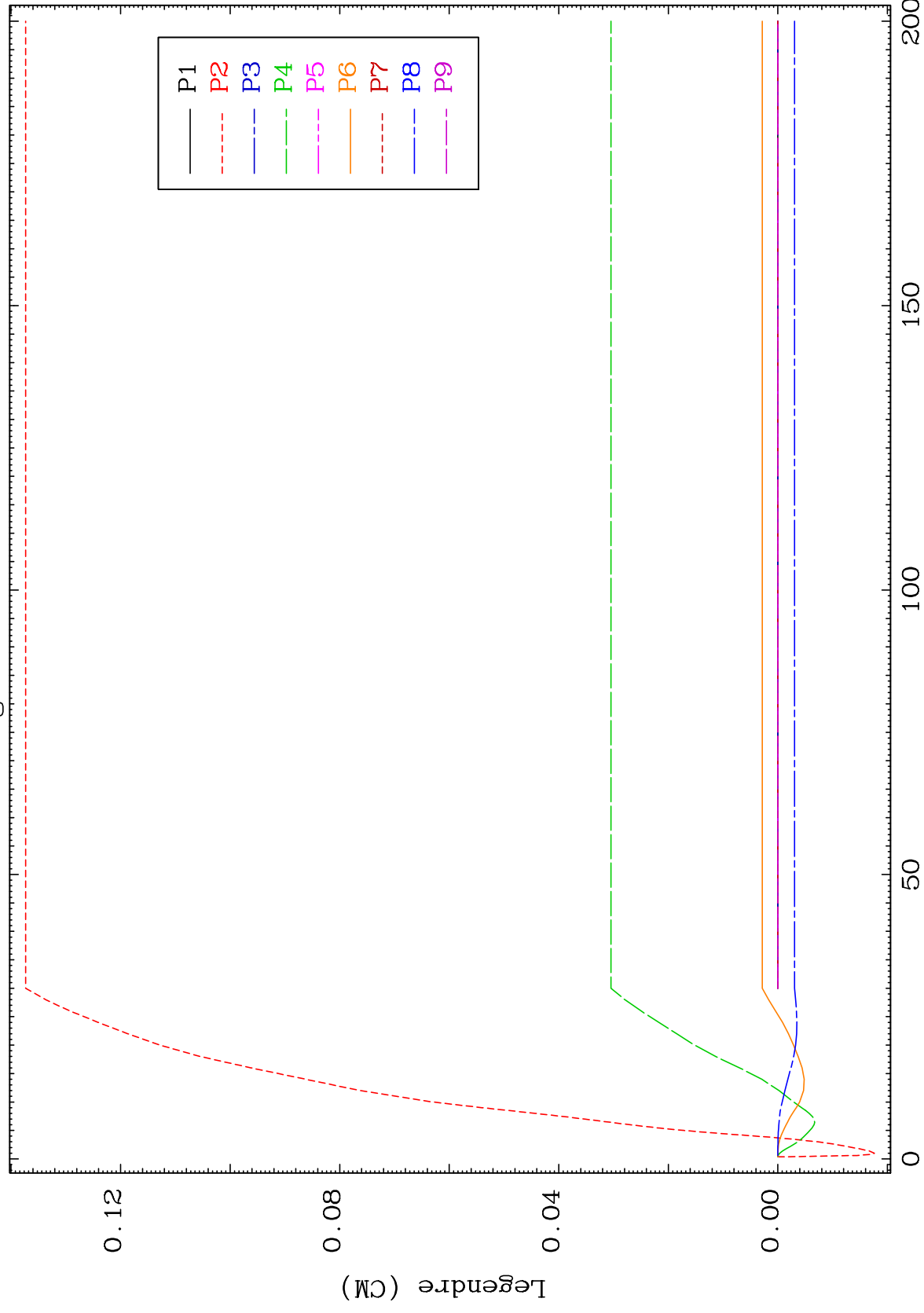




MAT 7528

MT= 66 (n, n') Level
Legendre Coefficients

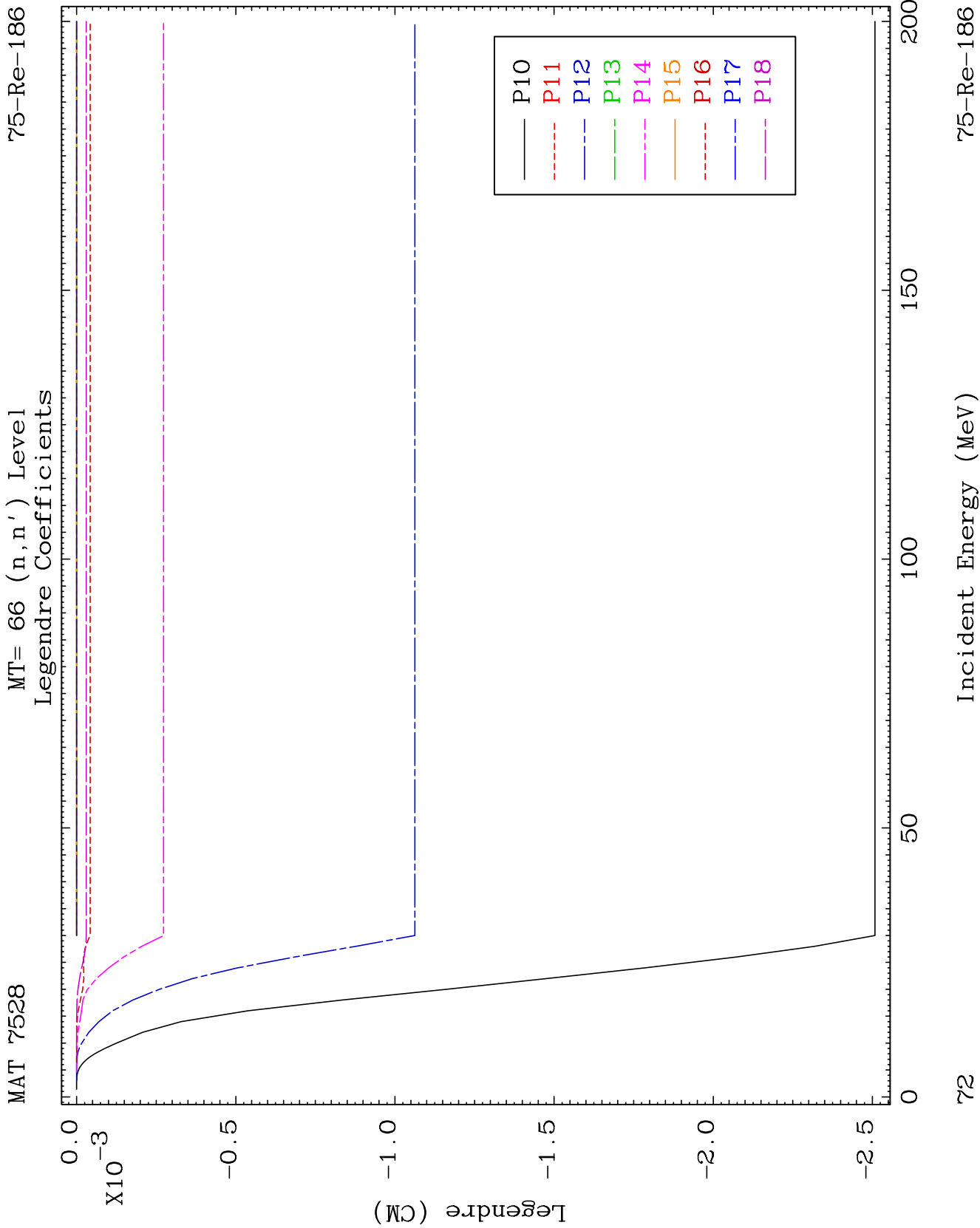
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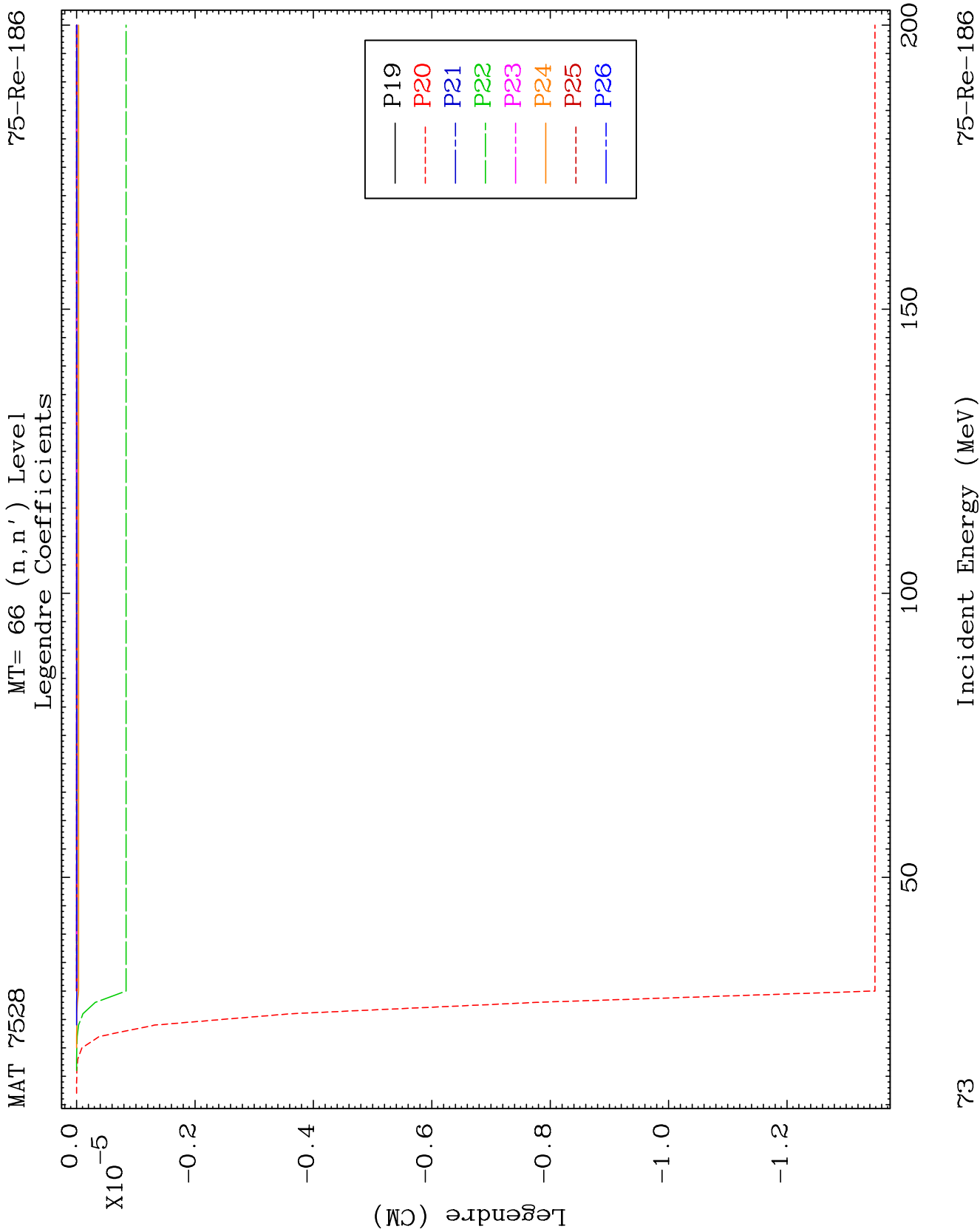


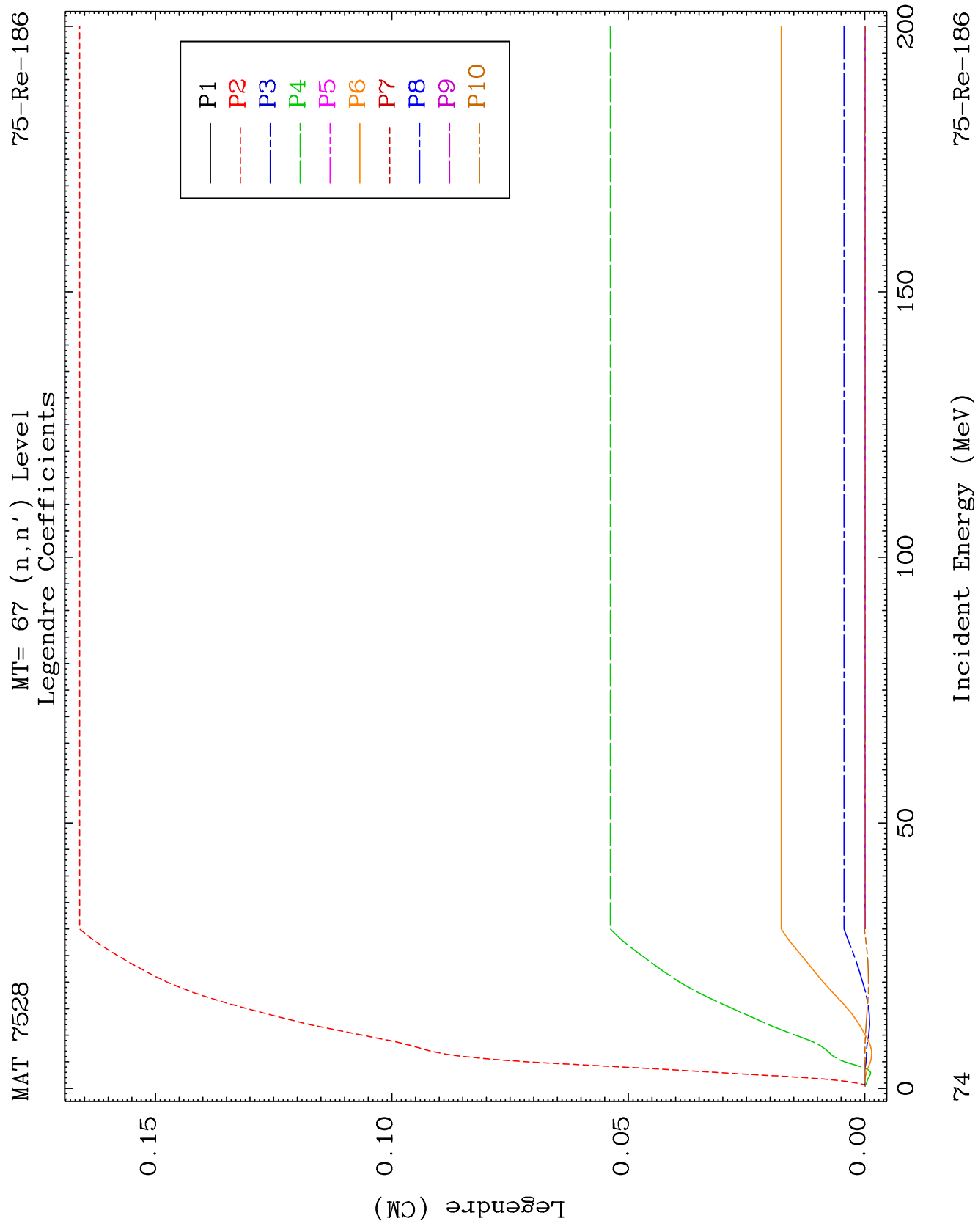
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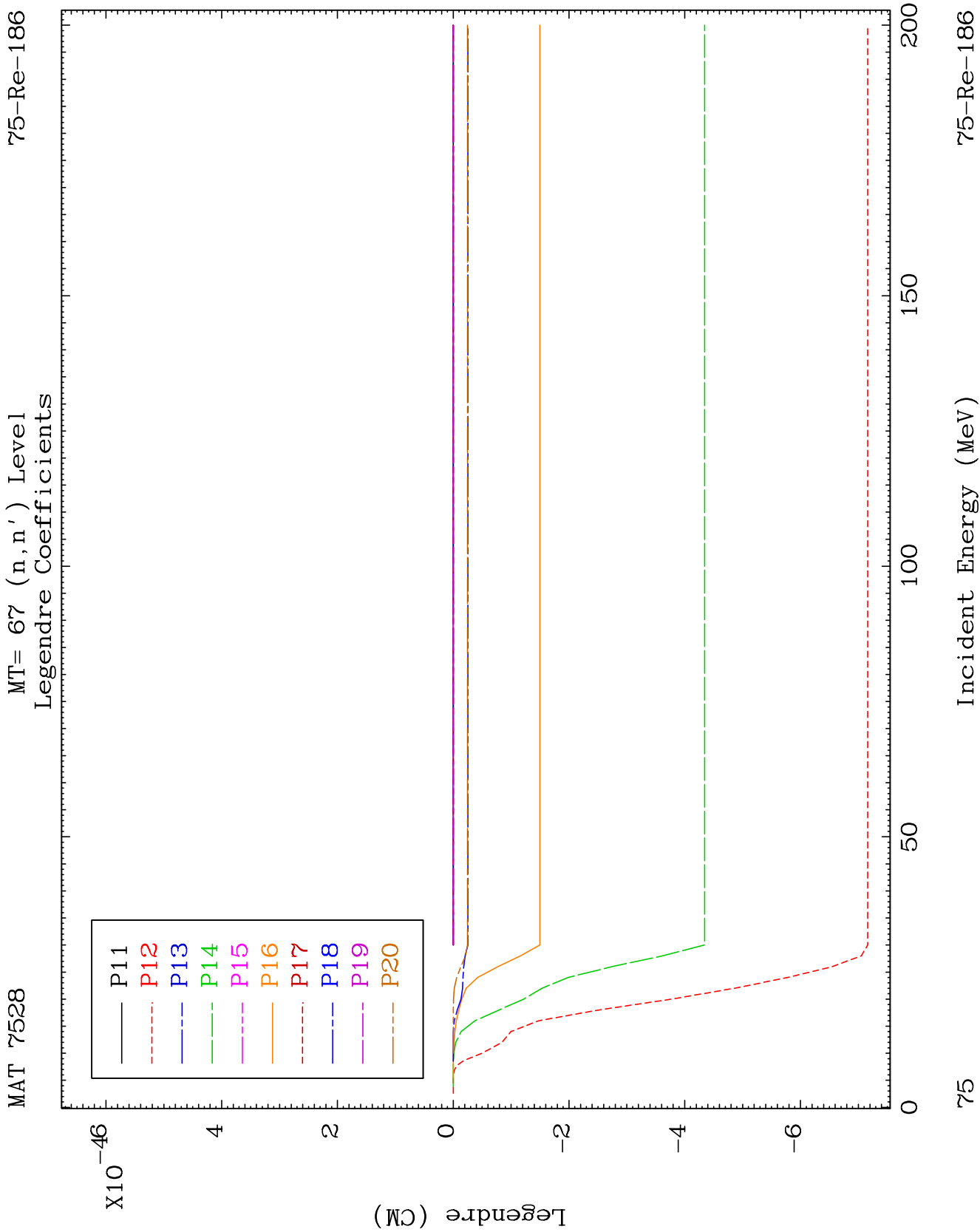
Incident Energy (MeV)

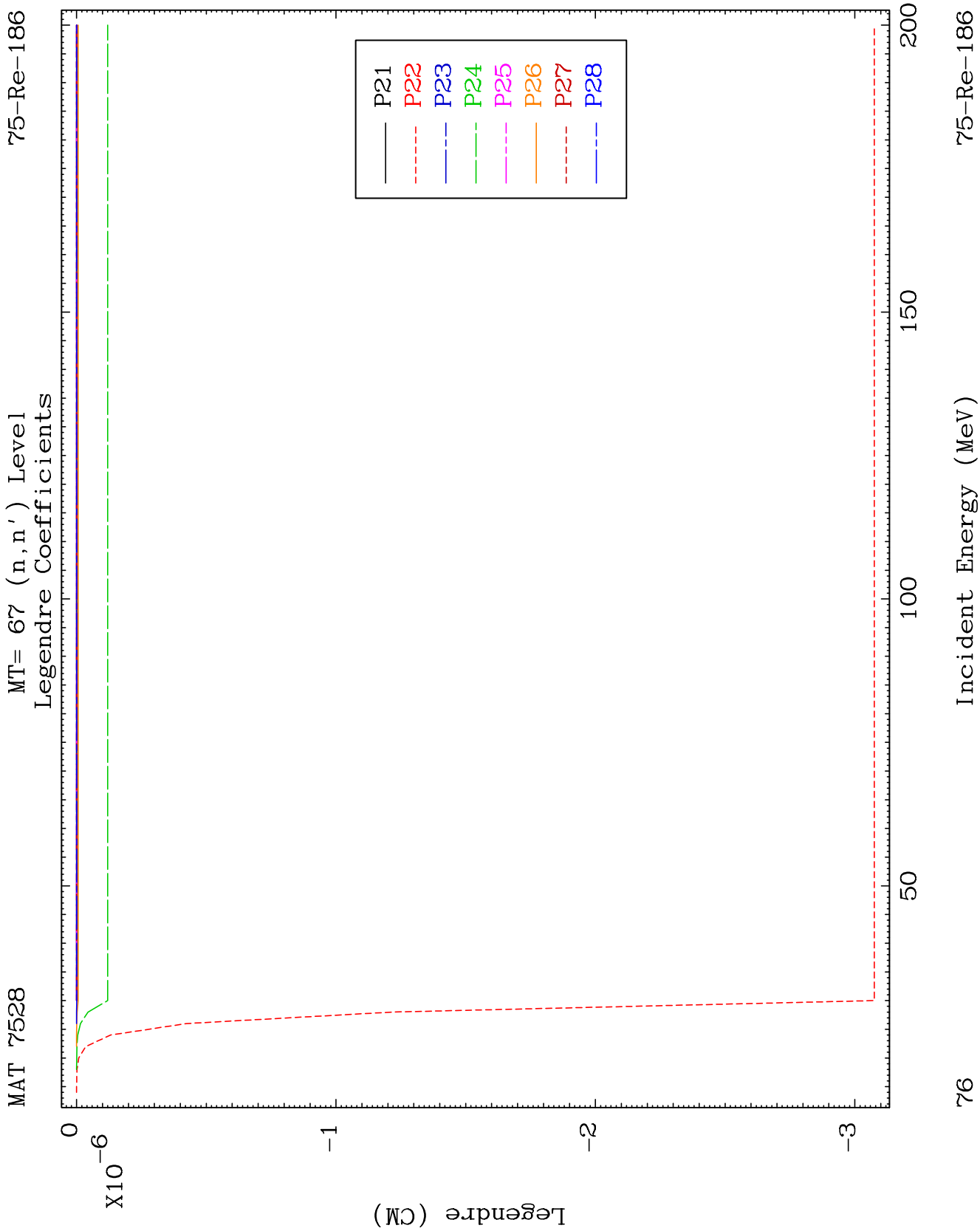
75-Re-186

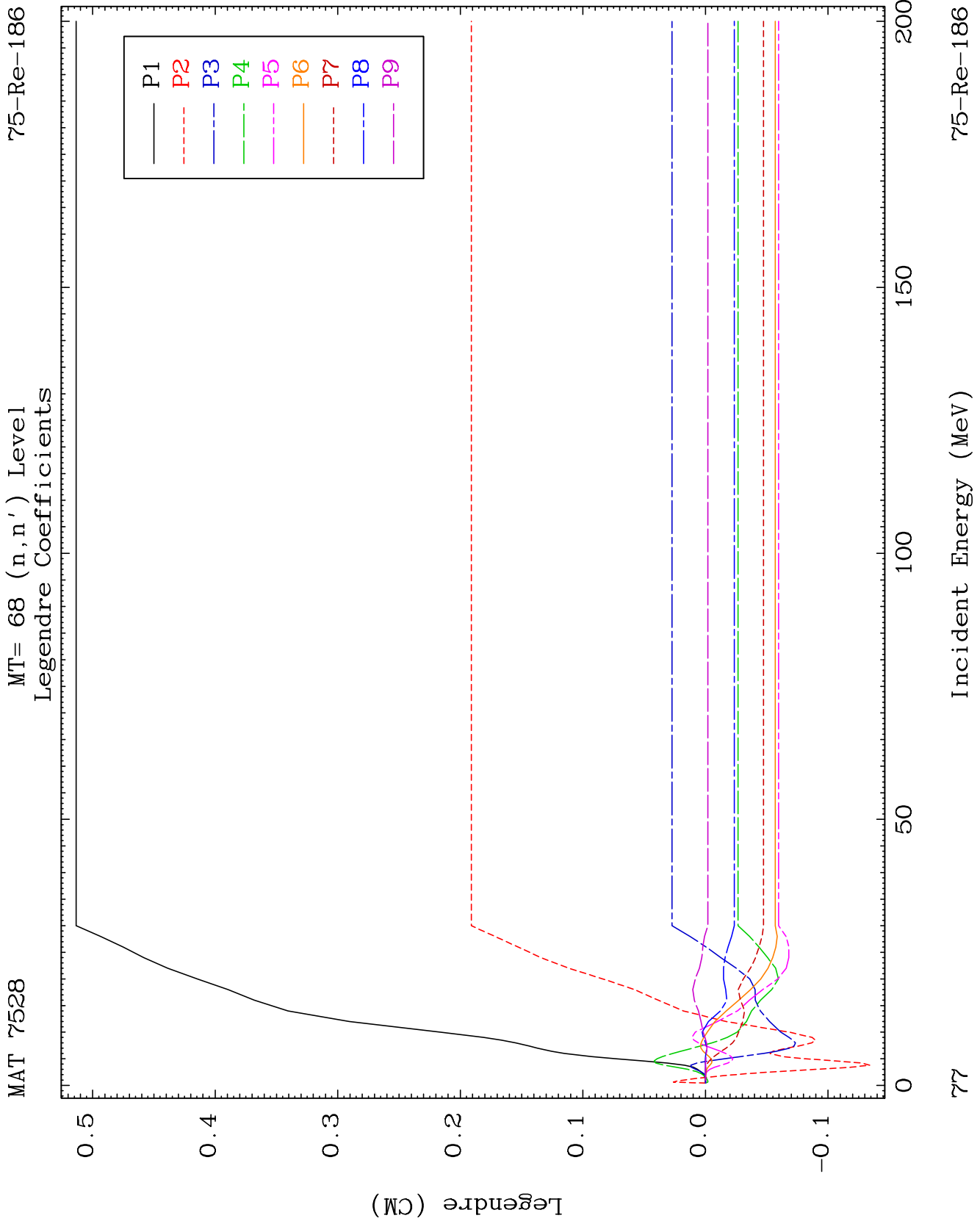


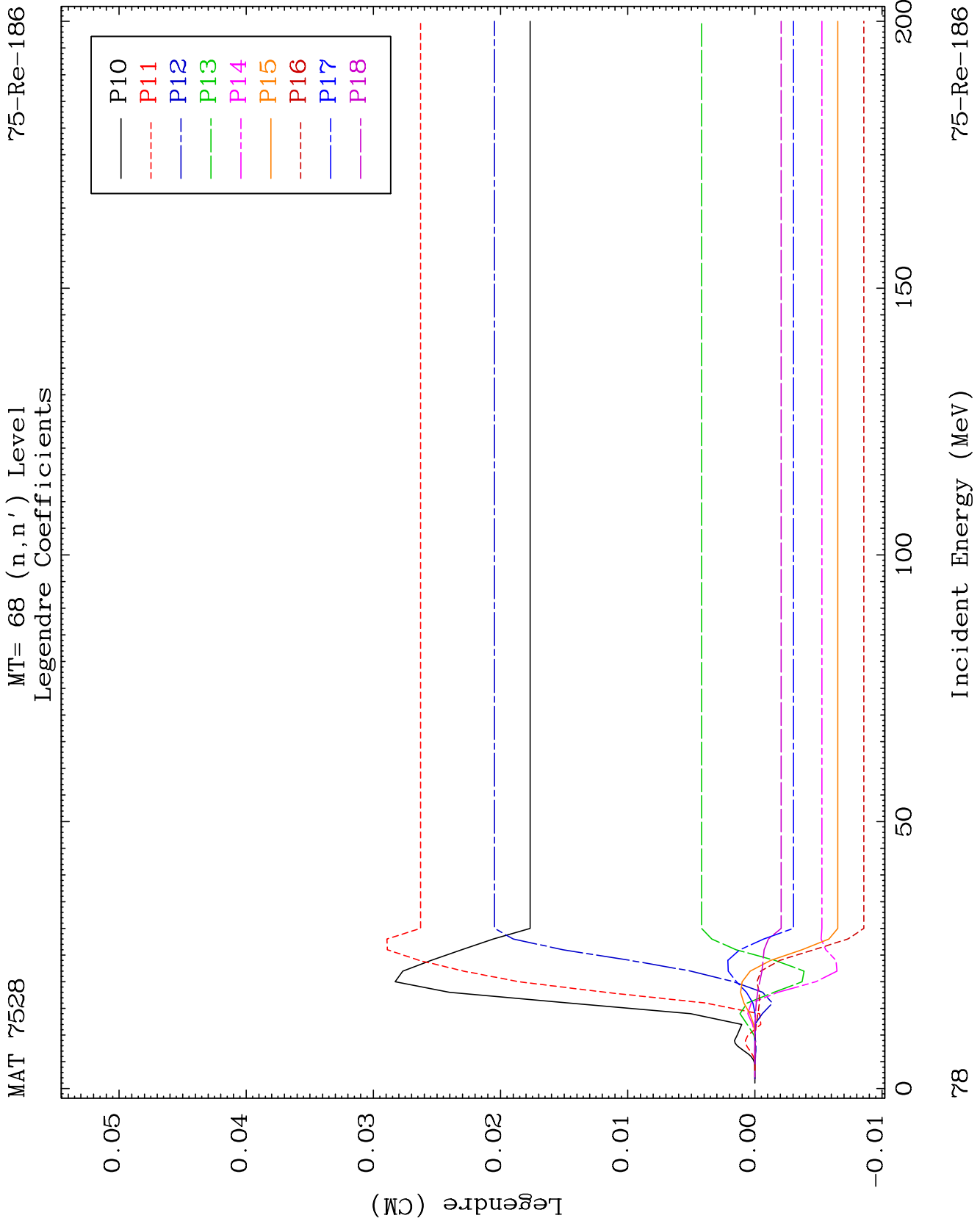


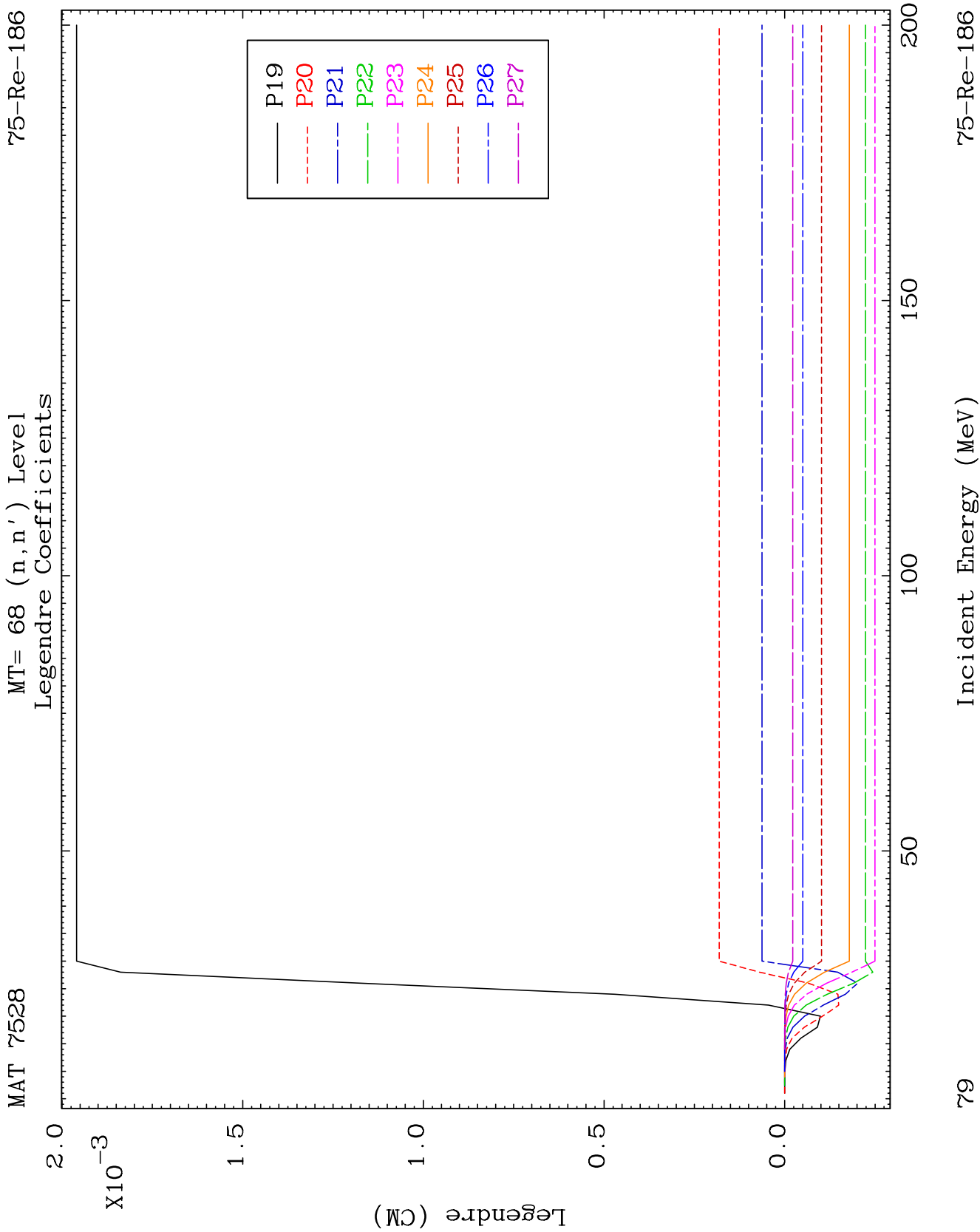










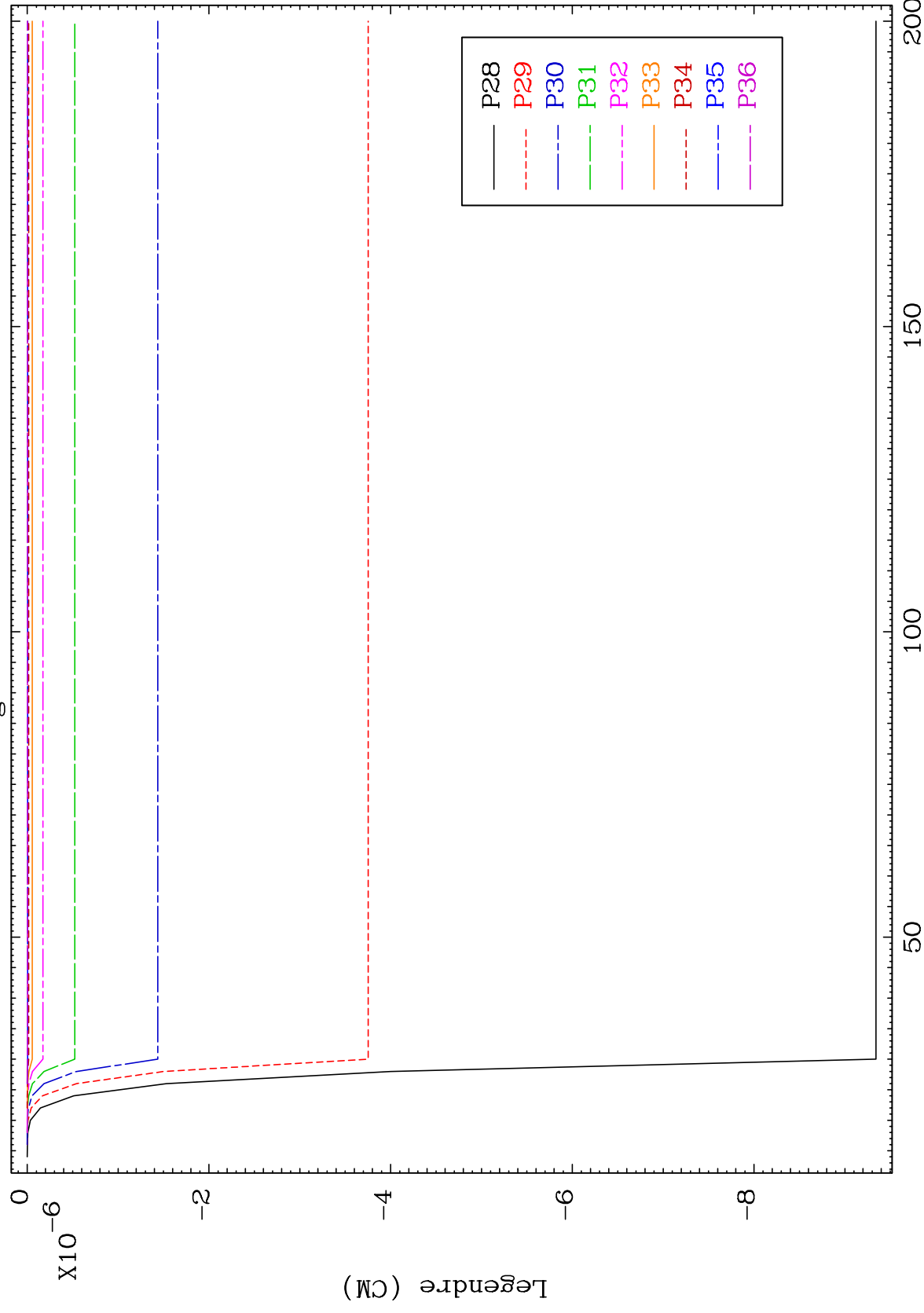


MAT 7528

MT= 68 (n, n') Level

75-Re-186

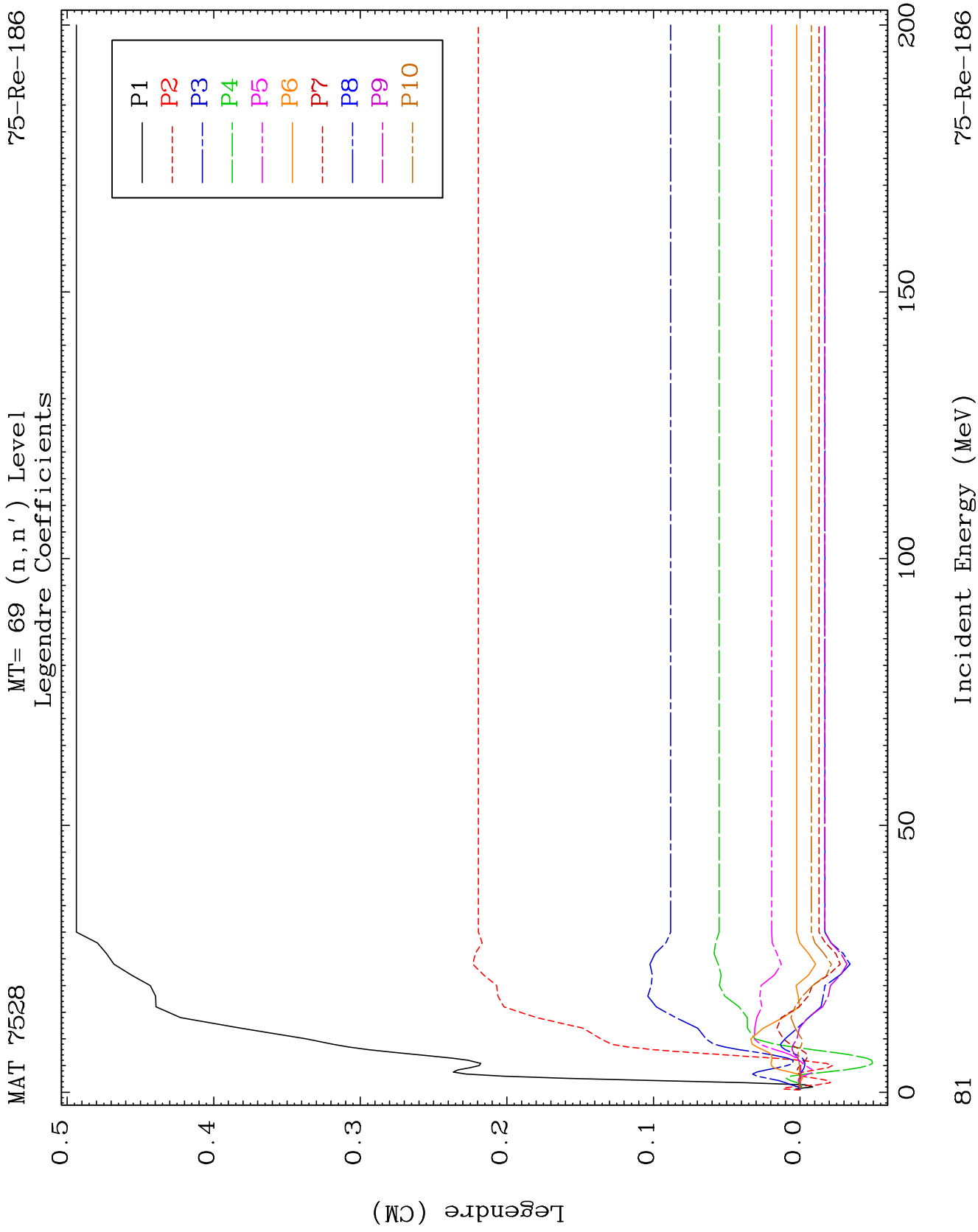
Legendre Coefficients

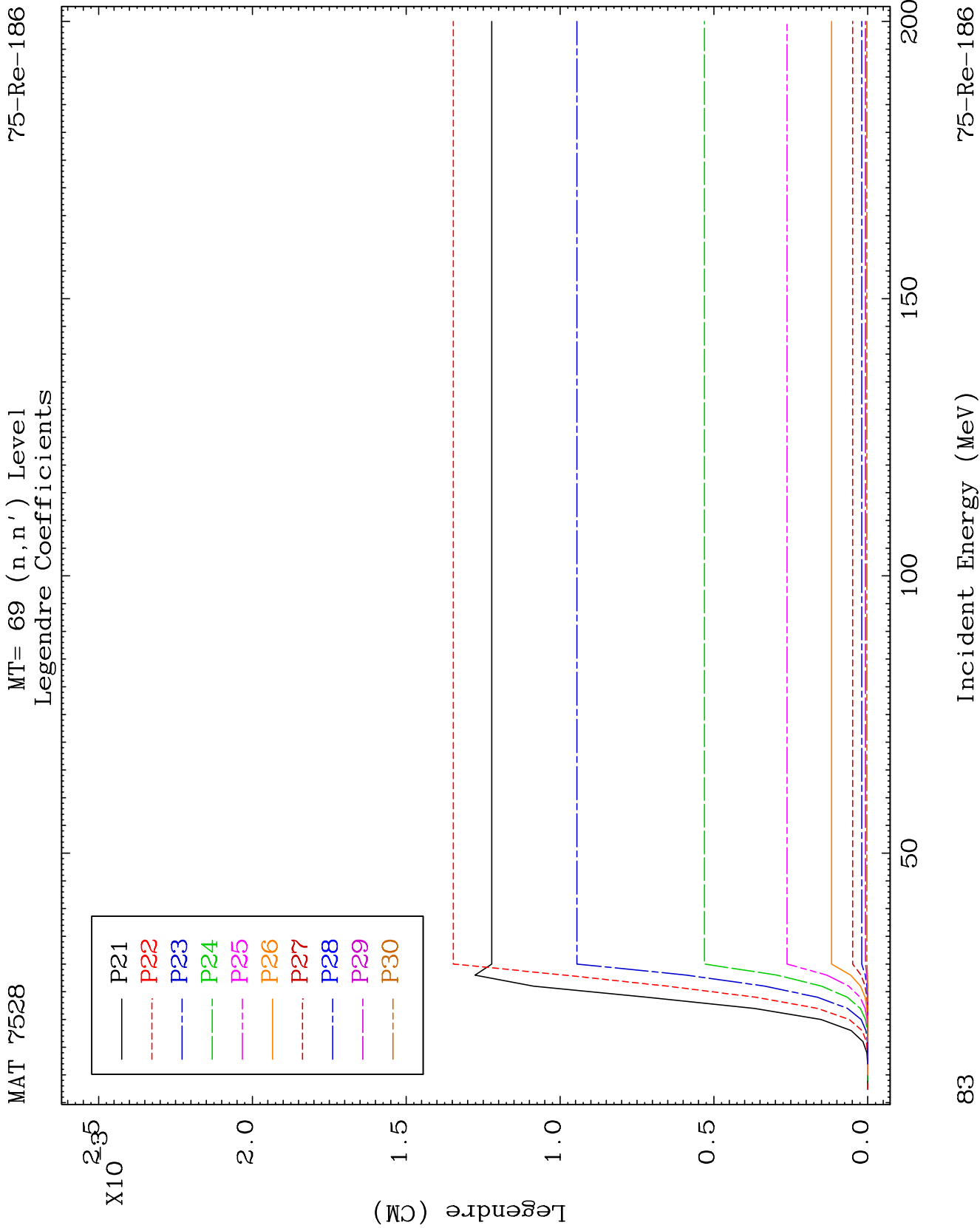


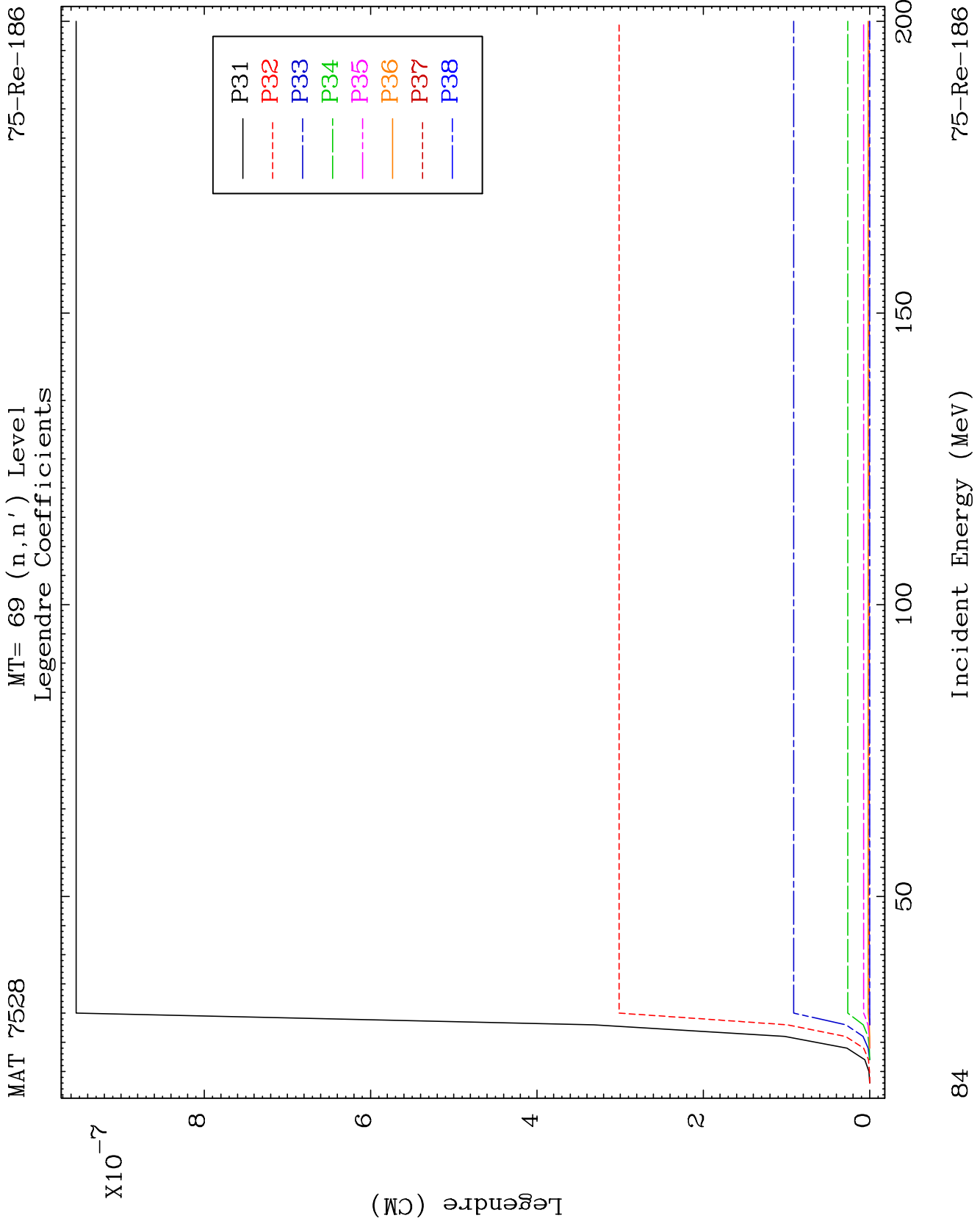
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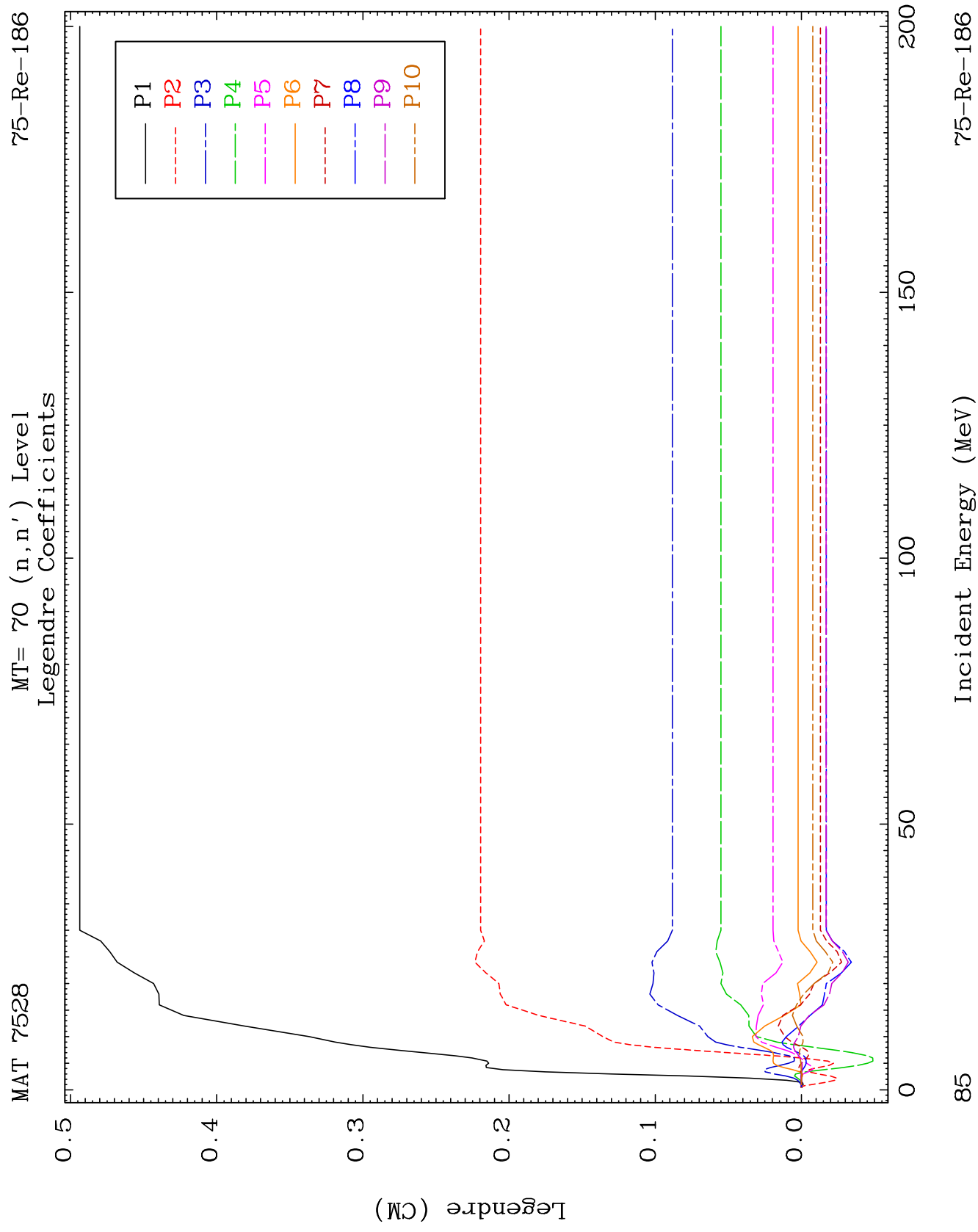
Incident Energy (MeV)

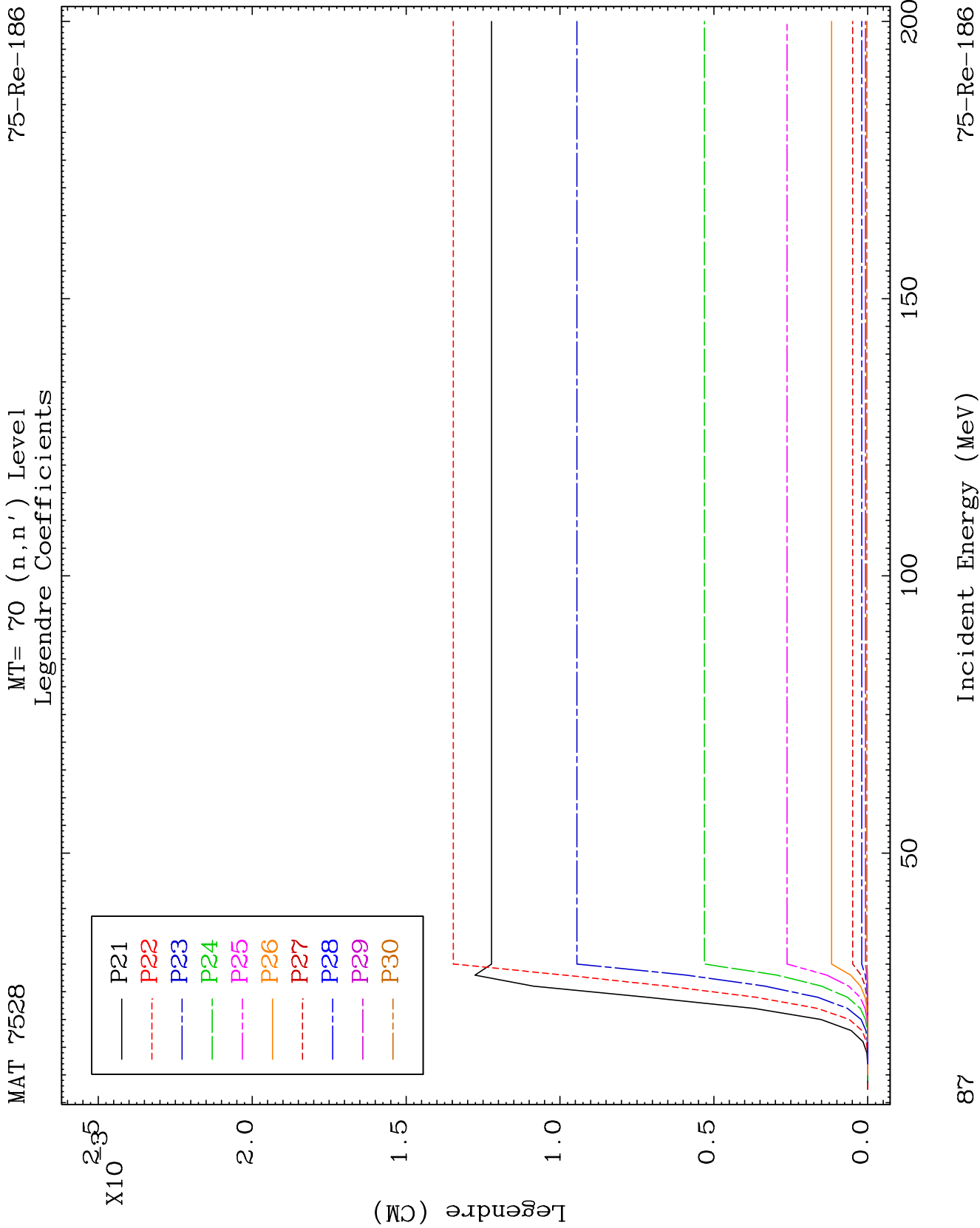
75-Re-186

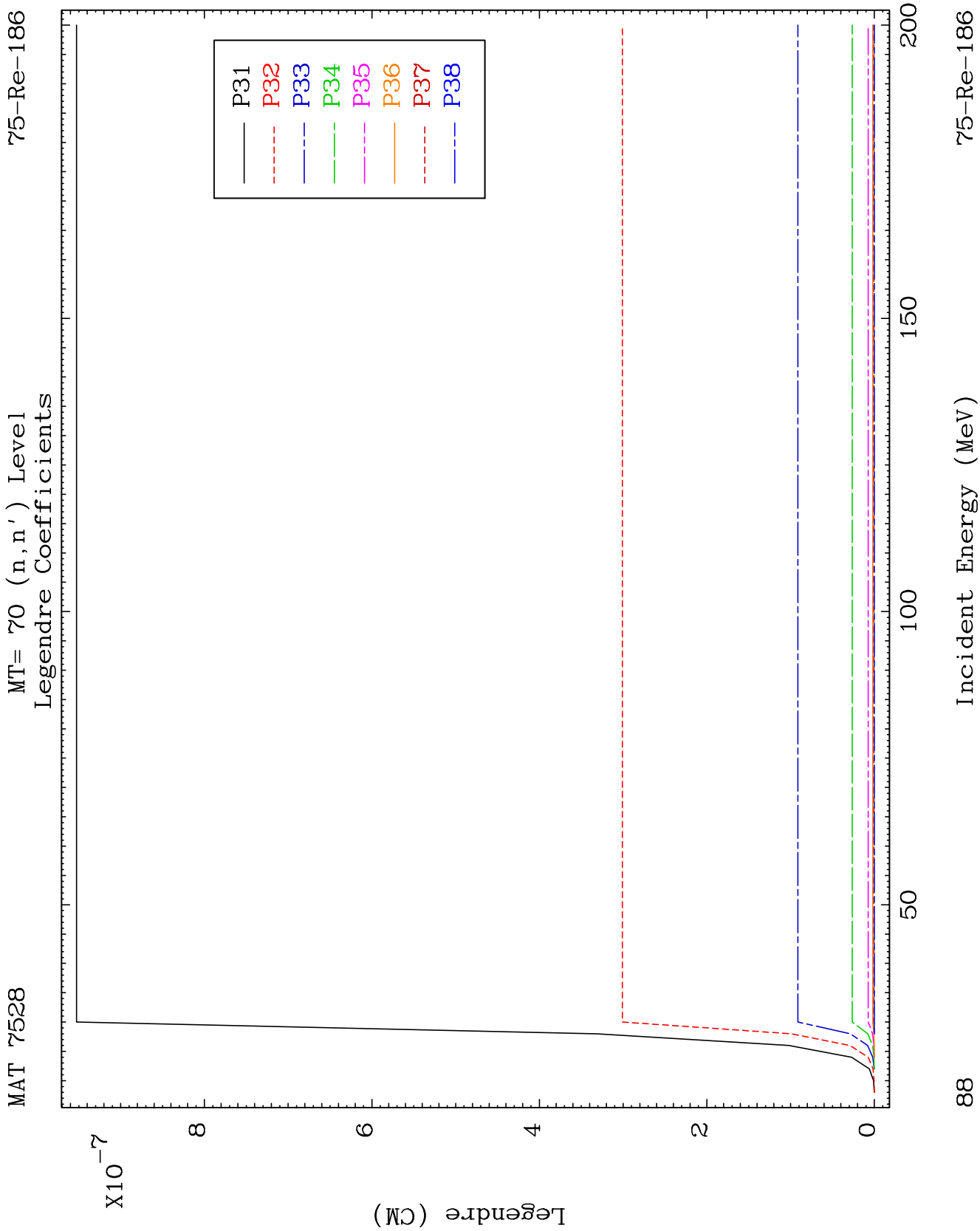


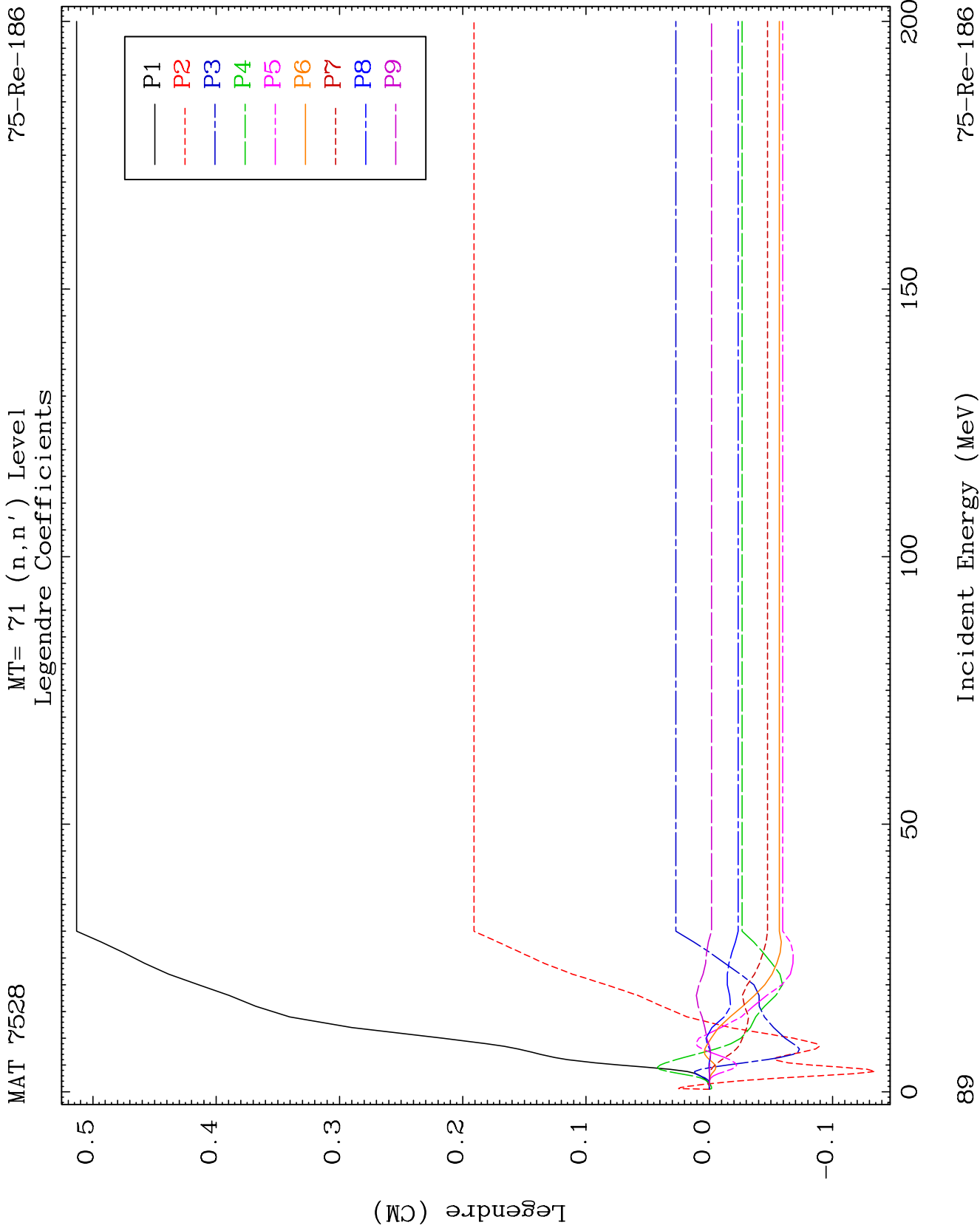


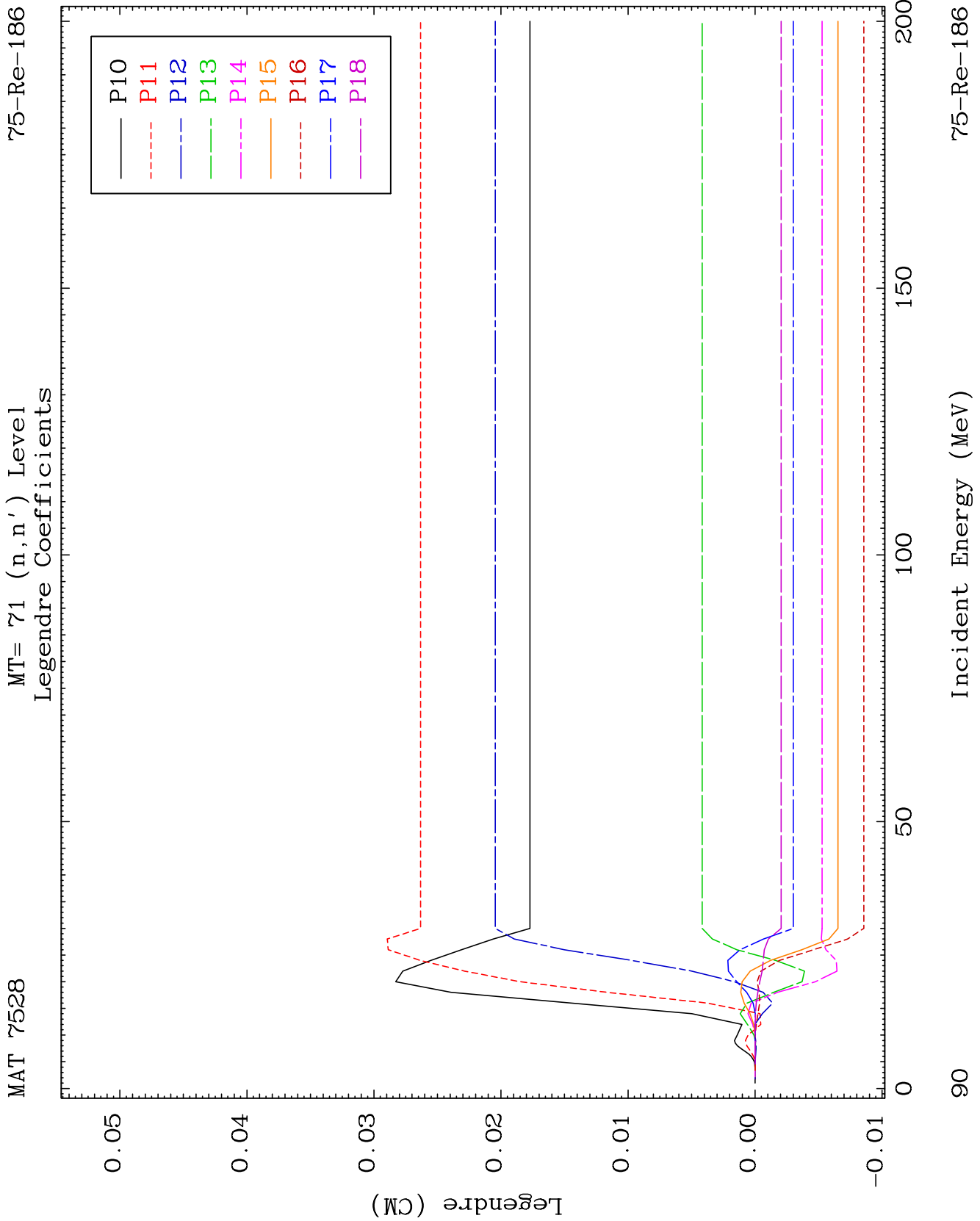


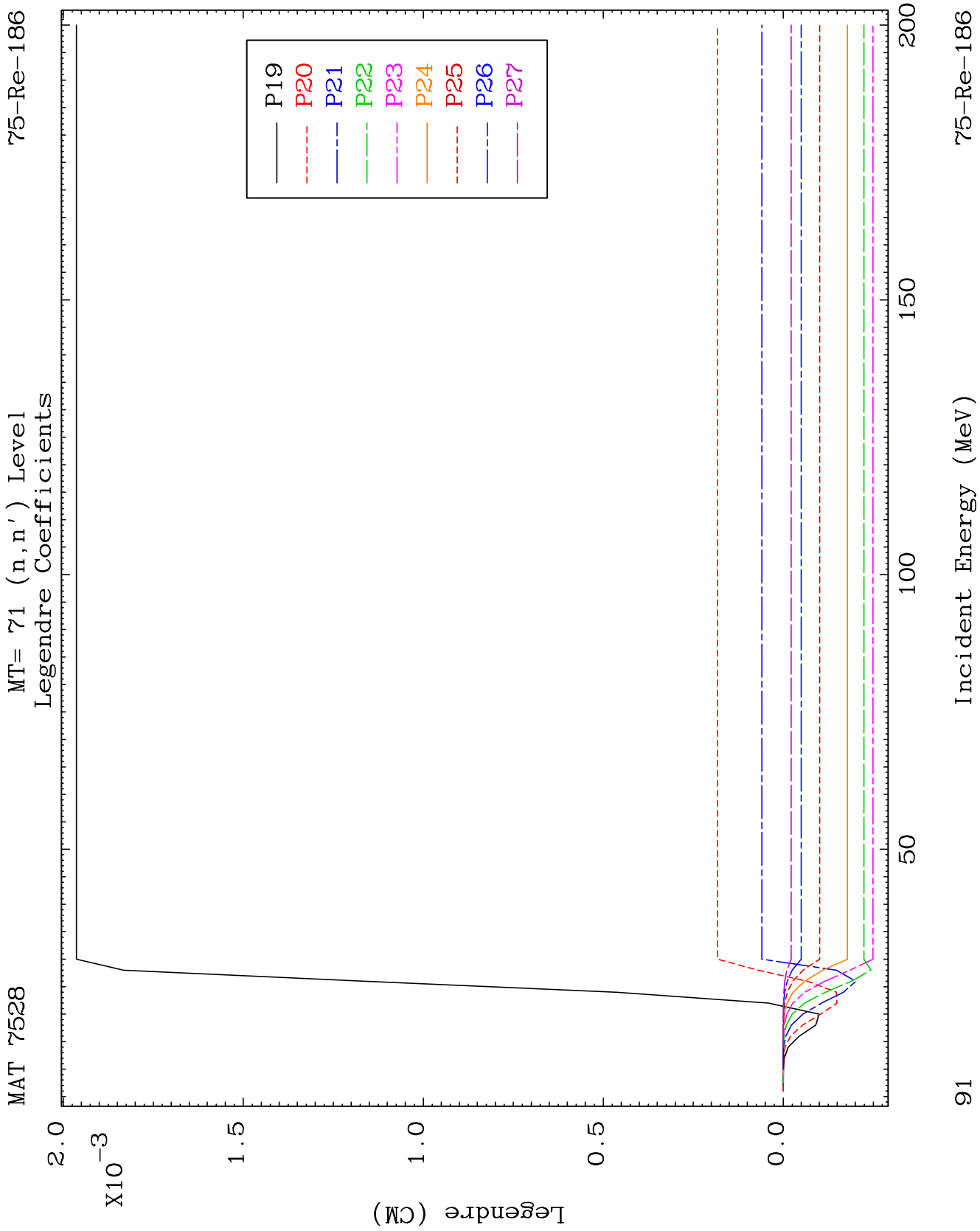


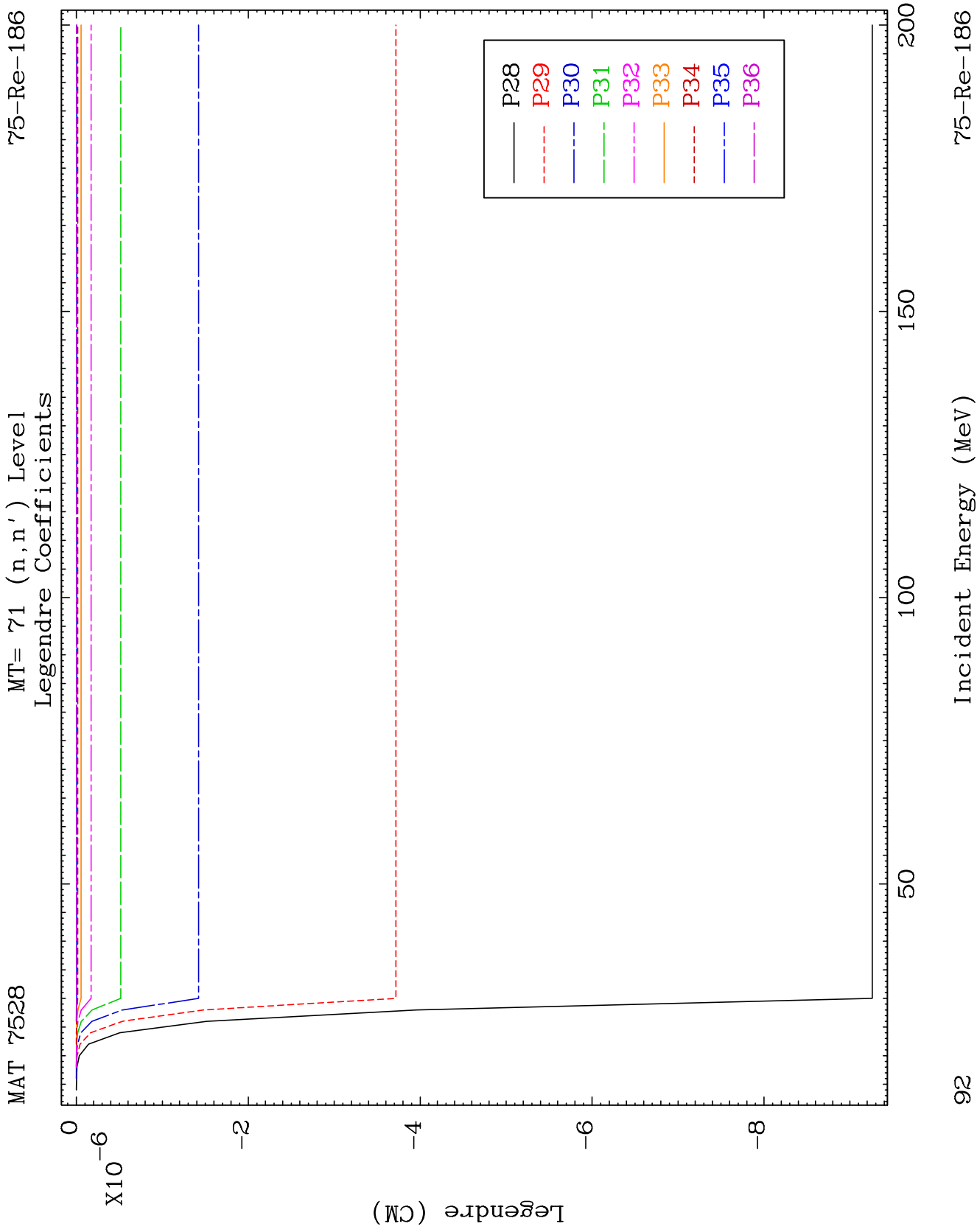


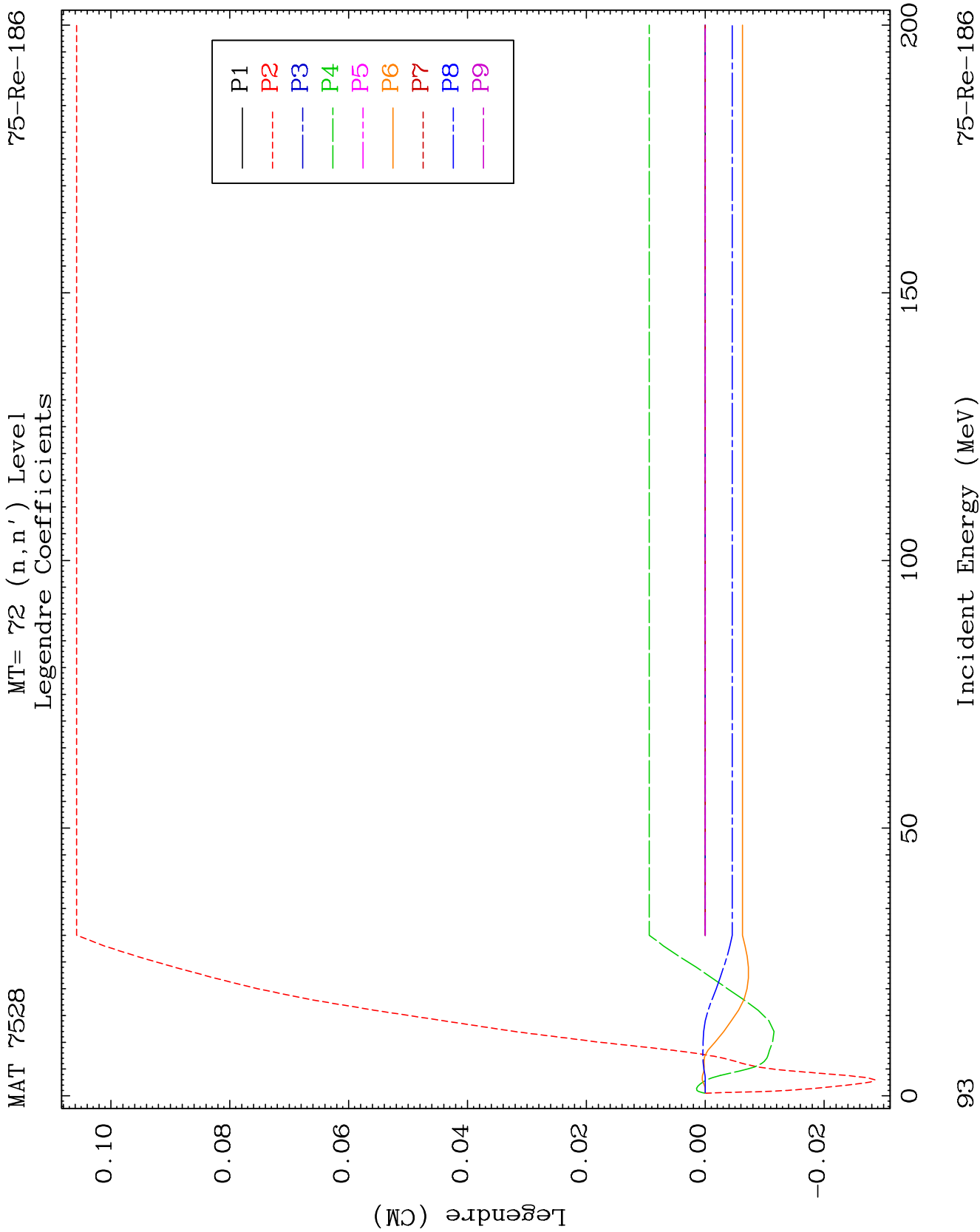


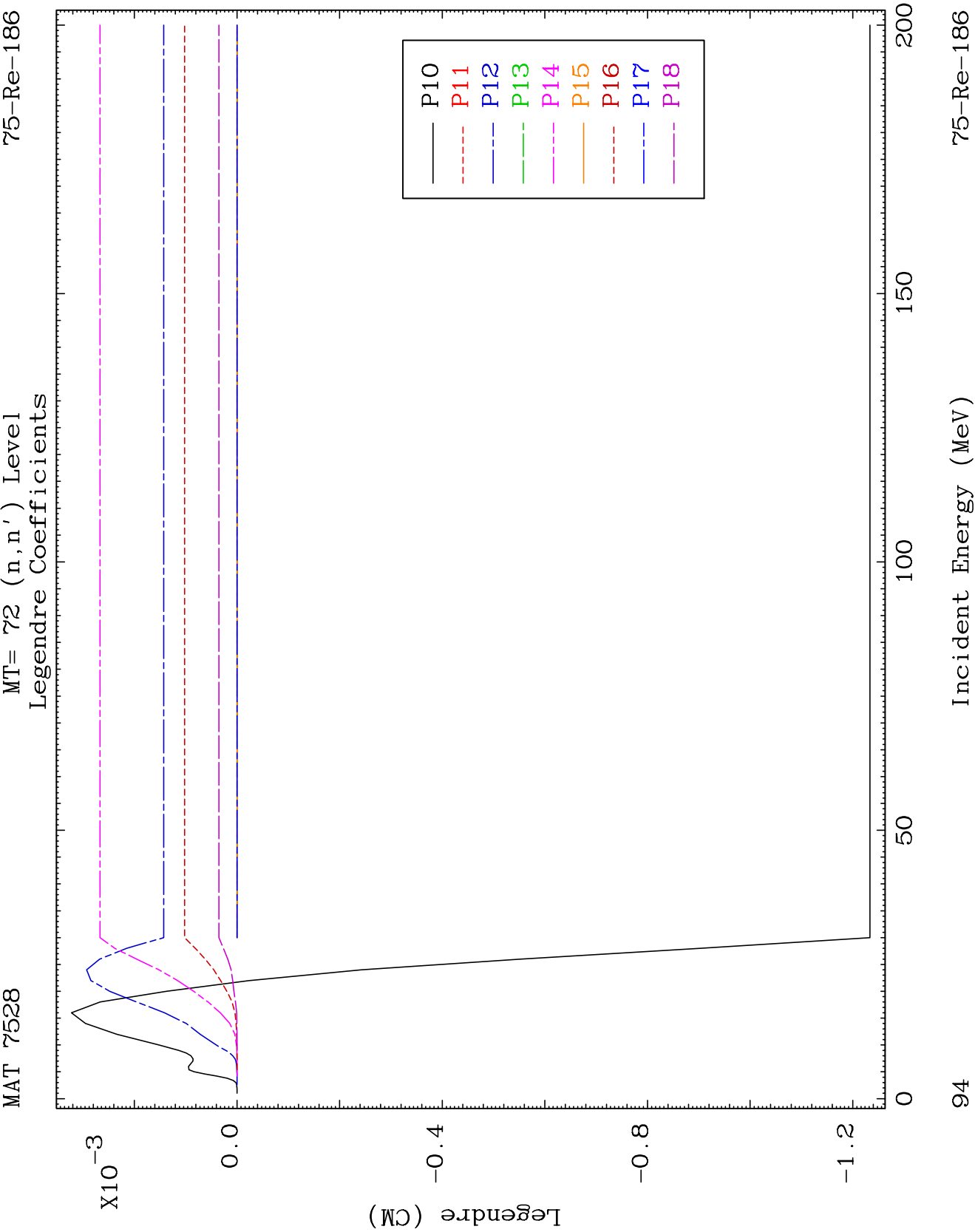








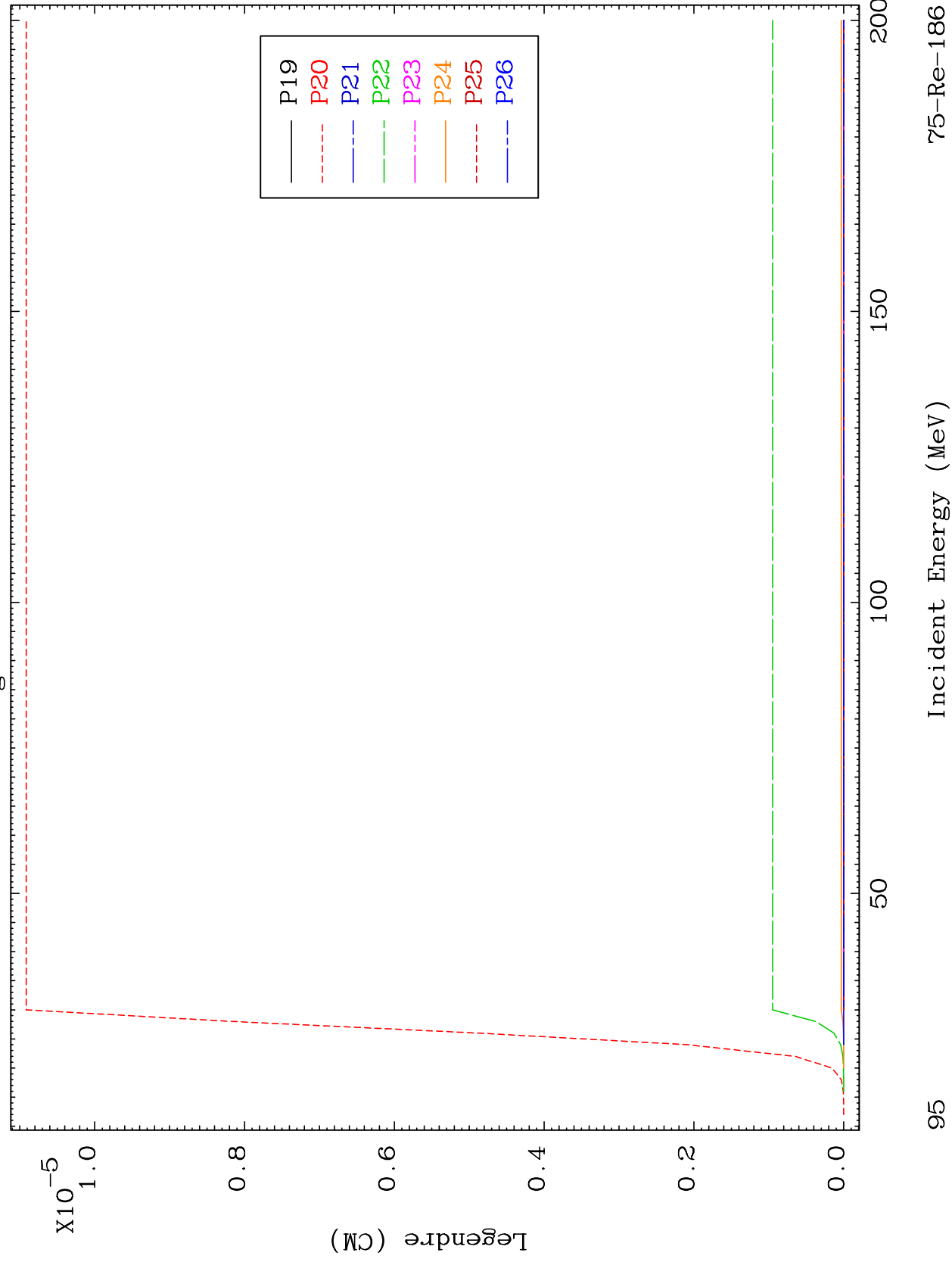




MAT 7528

MT= 72 (n, n') Level
Legendre Coefficients

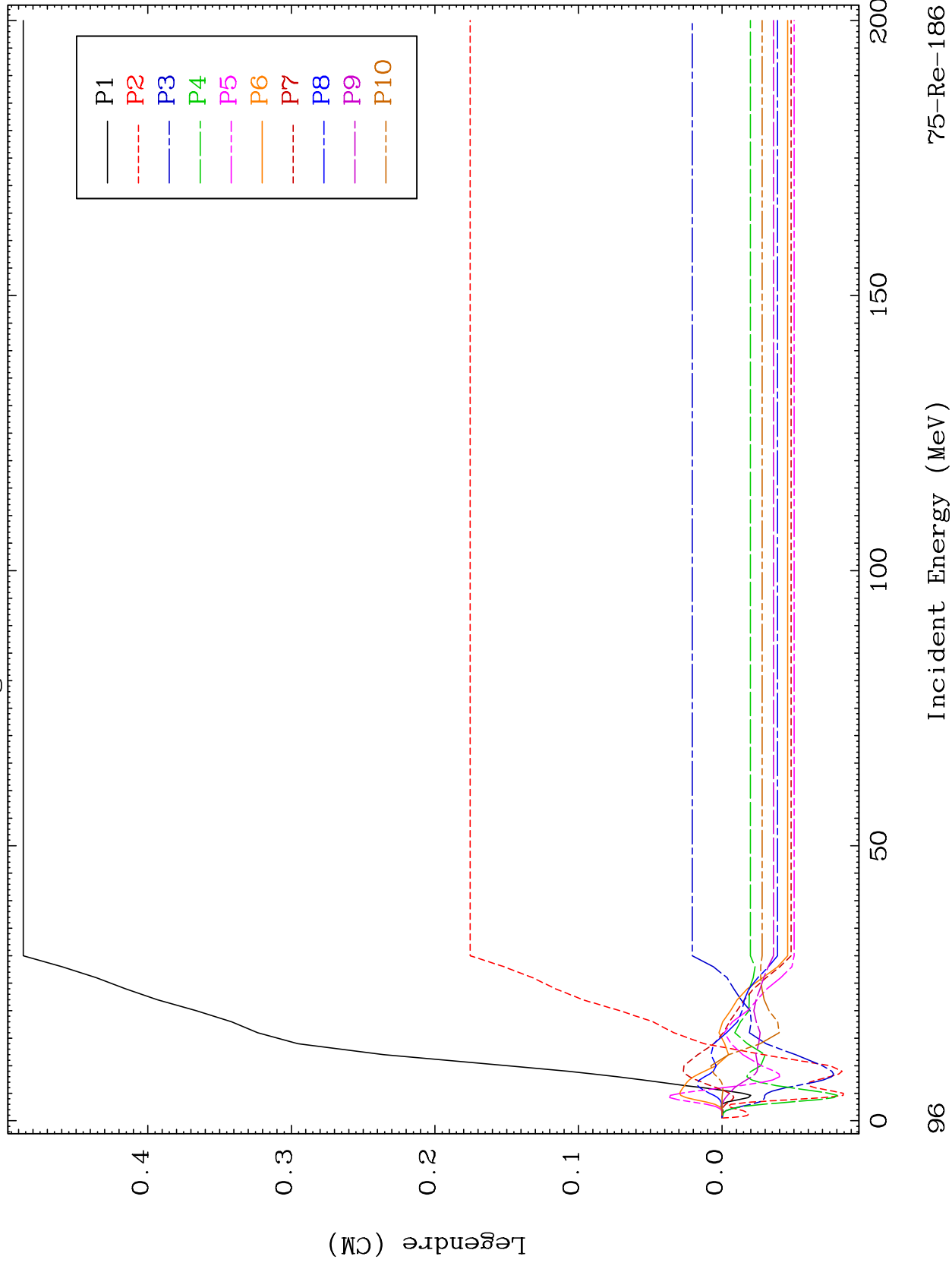
75-Re-186



MAT 7528

MT= 73 (n, n') Level
Legendre Coefficients

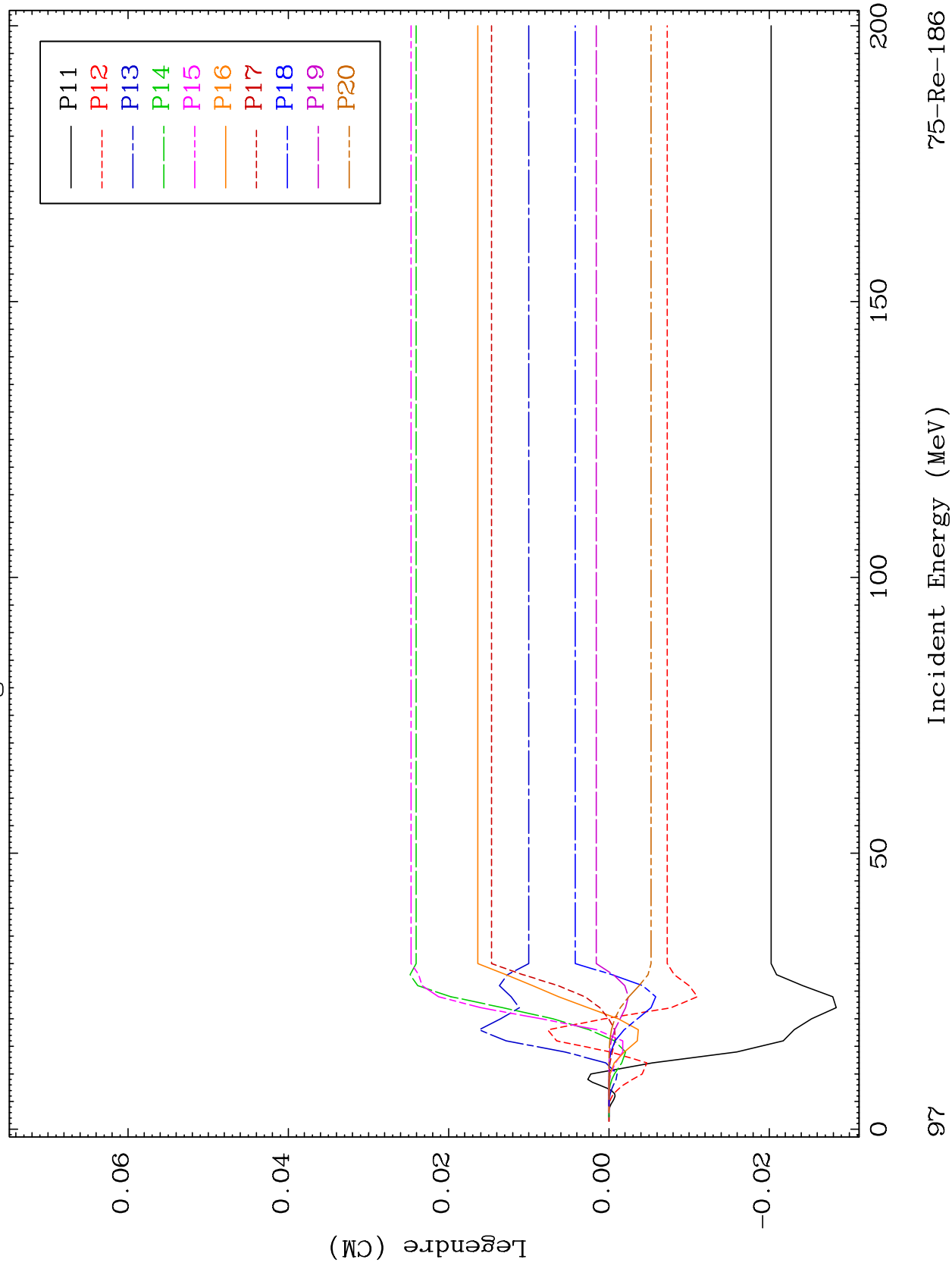
75-Re-186

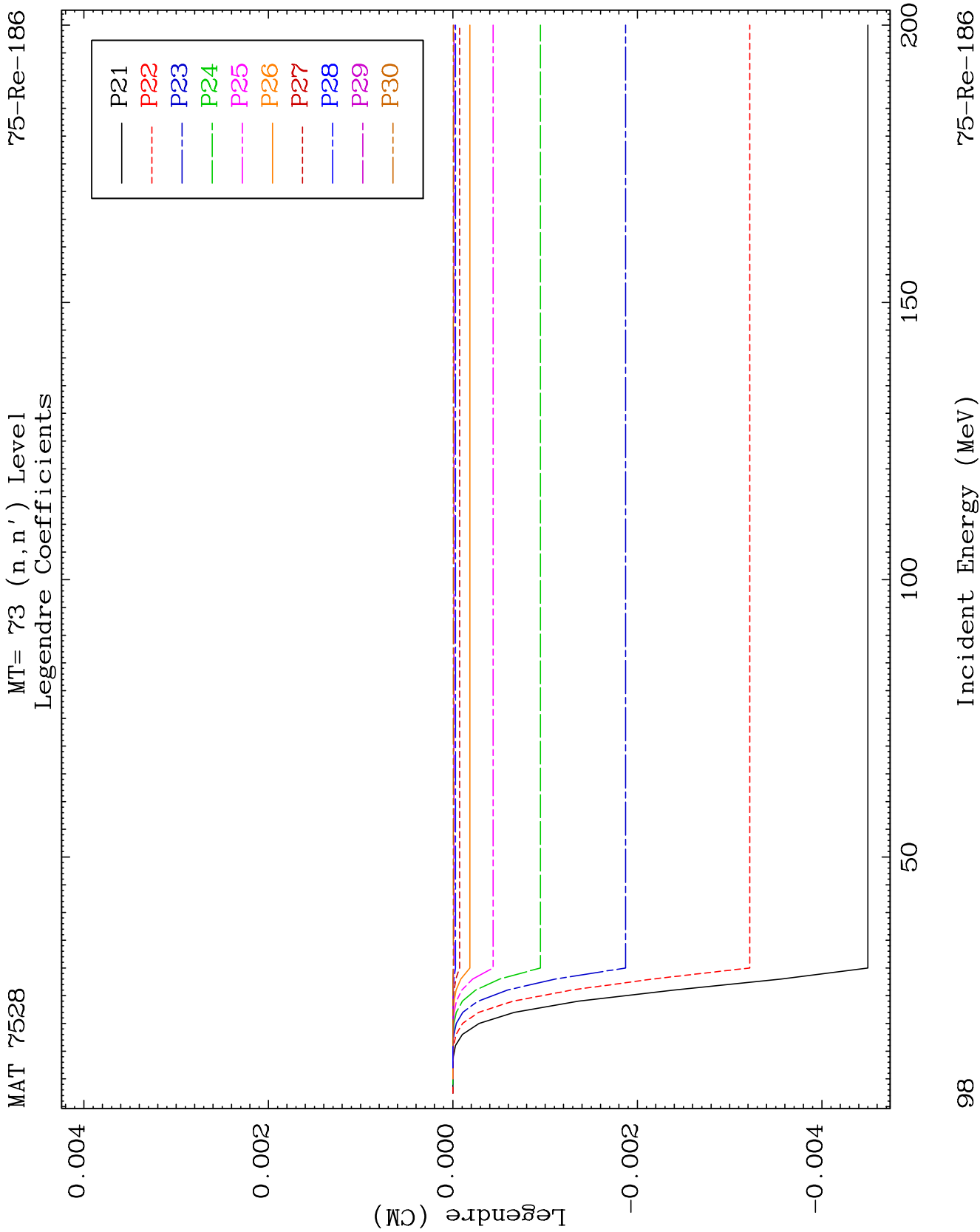


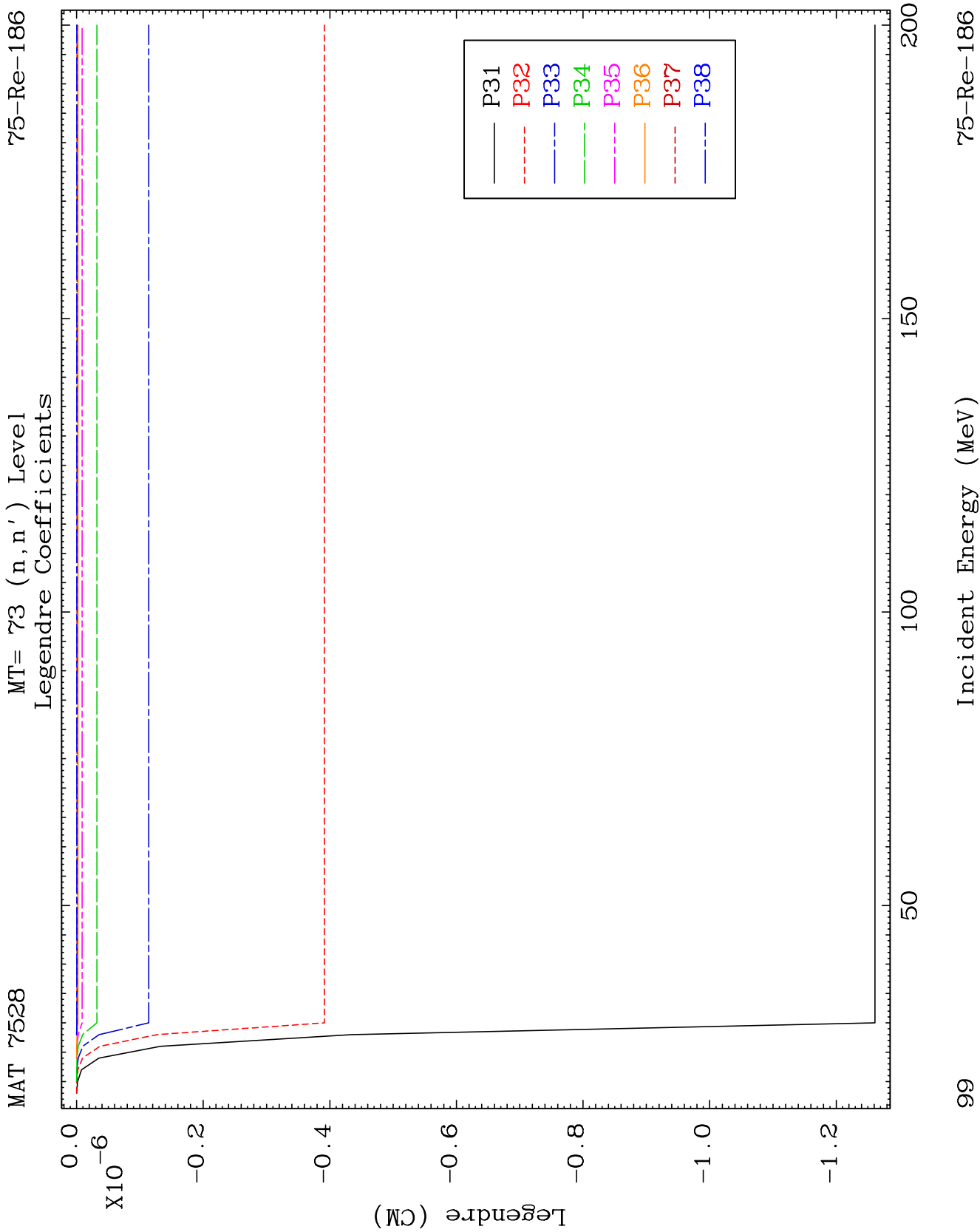
MAT 7528

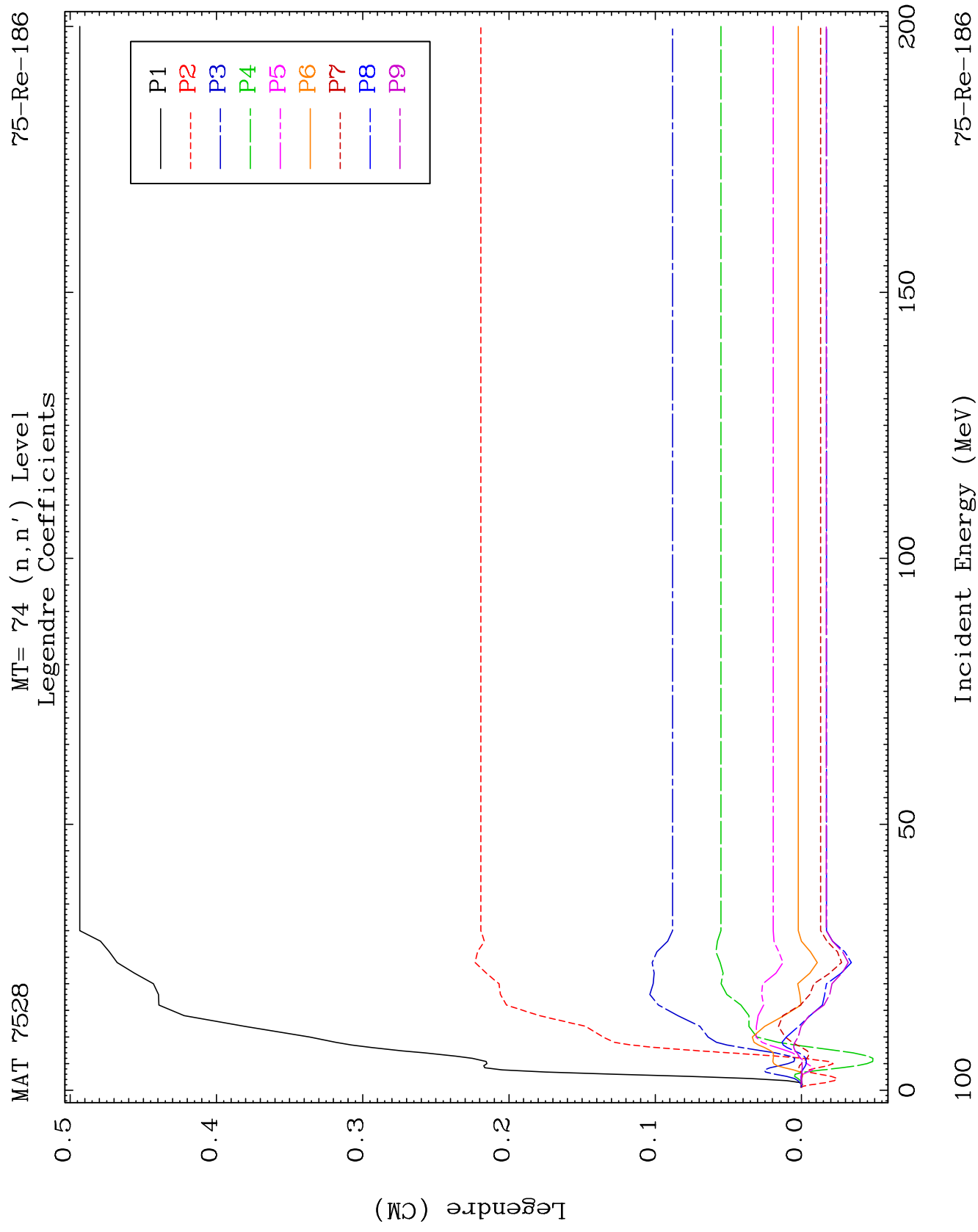
MT= 73 (n, n') Level
Legendre Coefficients

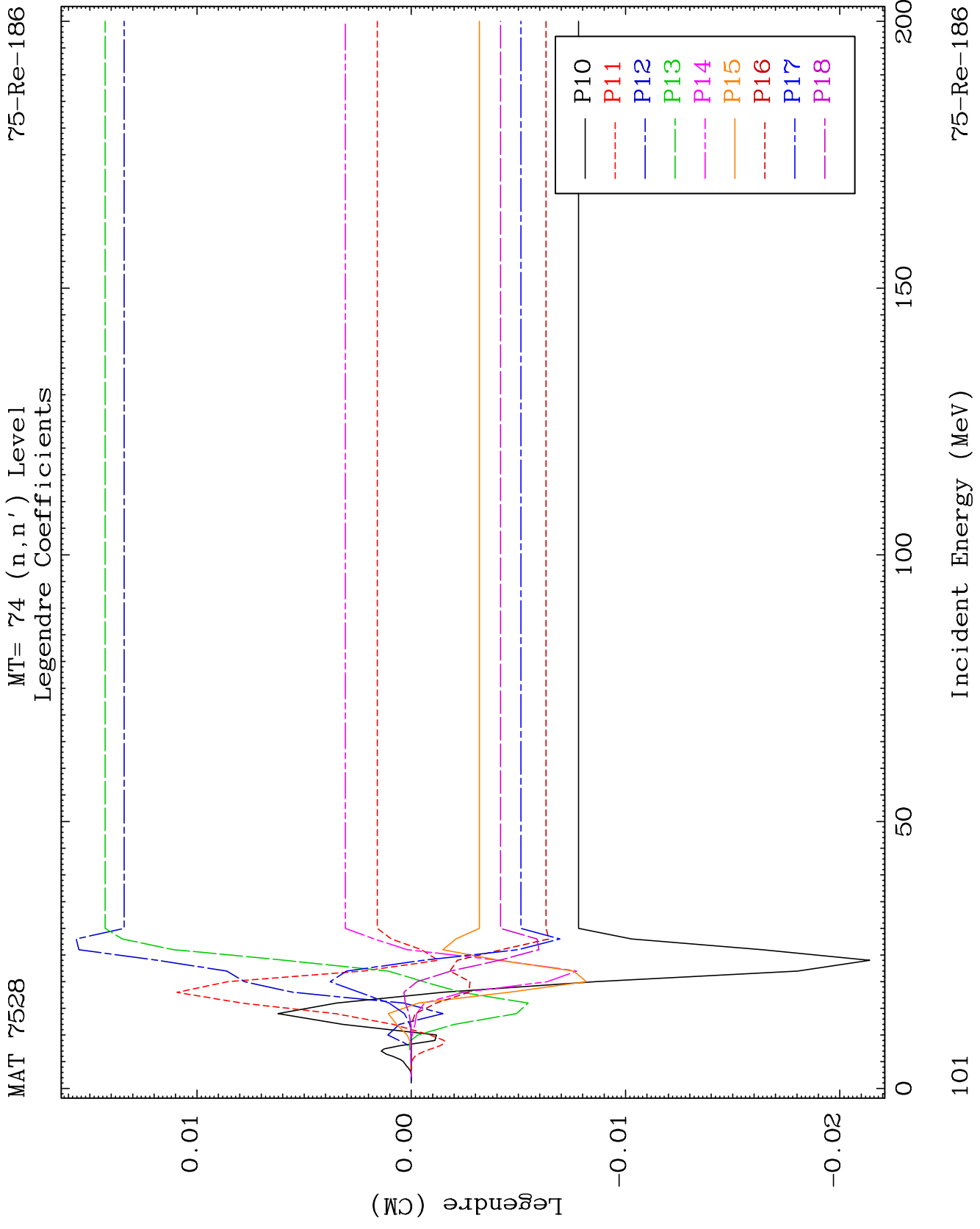
75-Re-186

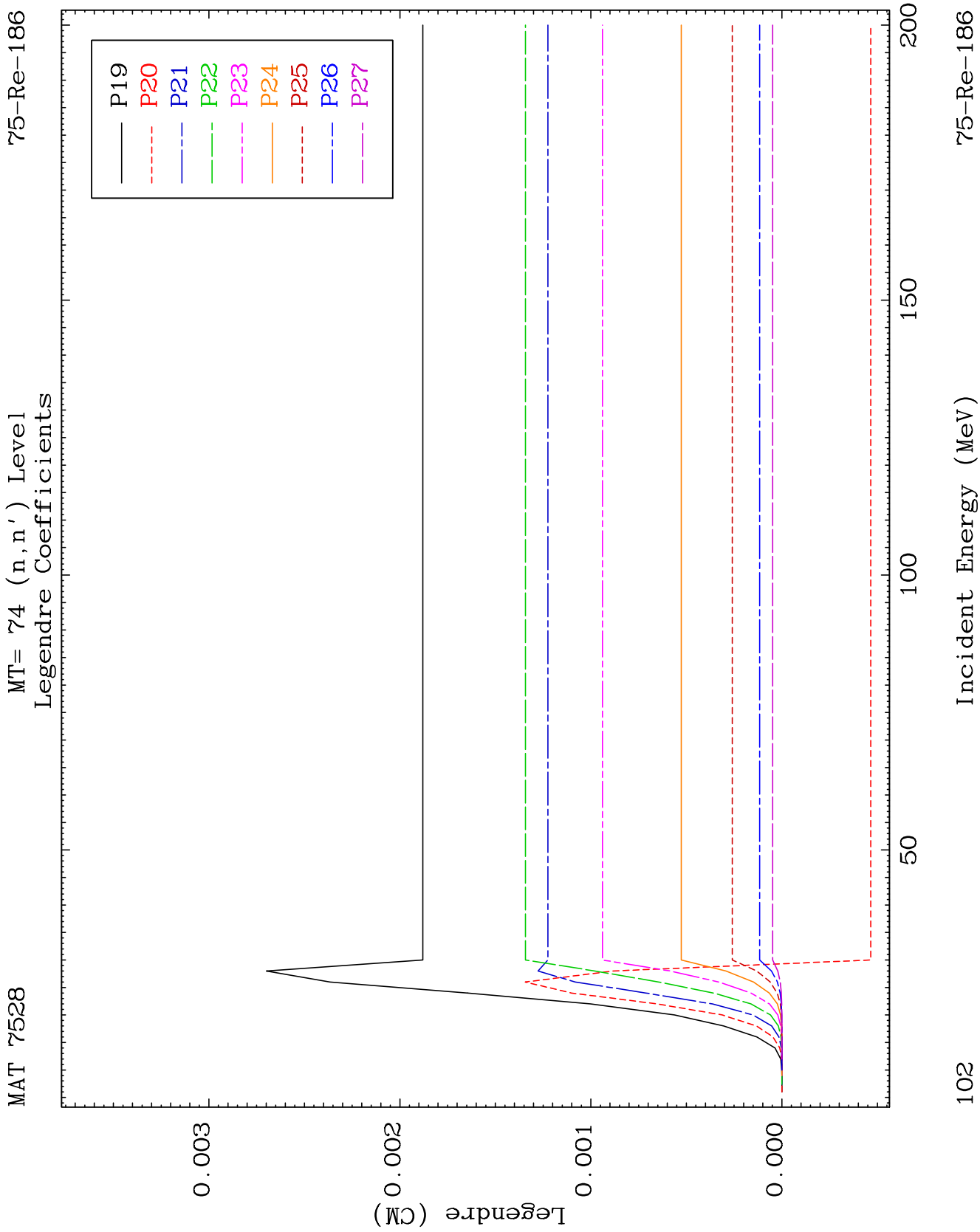








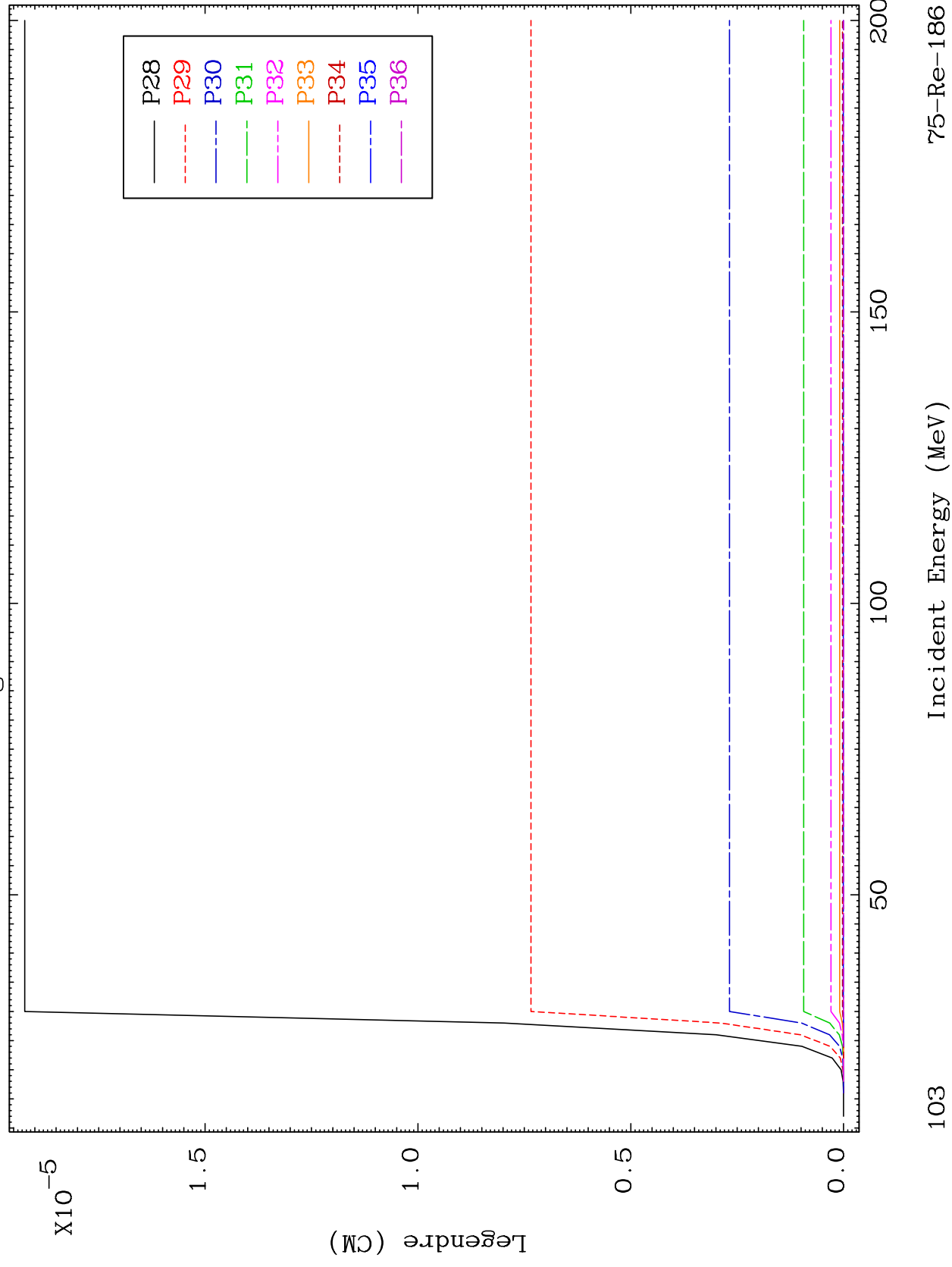




MAT 7528

MT= 74 (n, n') Level
Legendre Coefficients

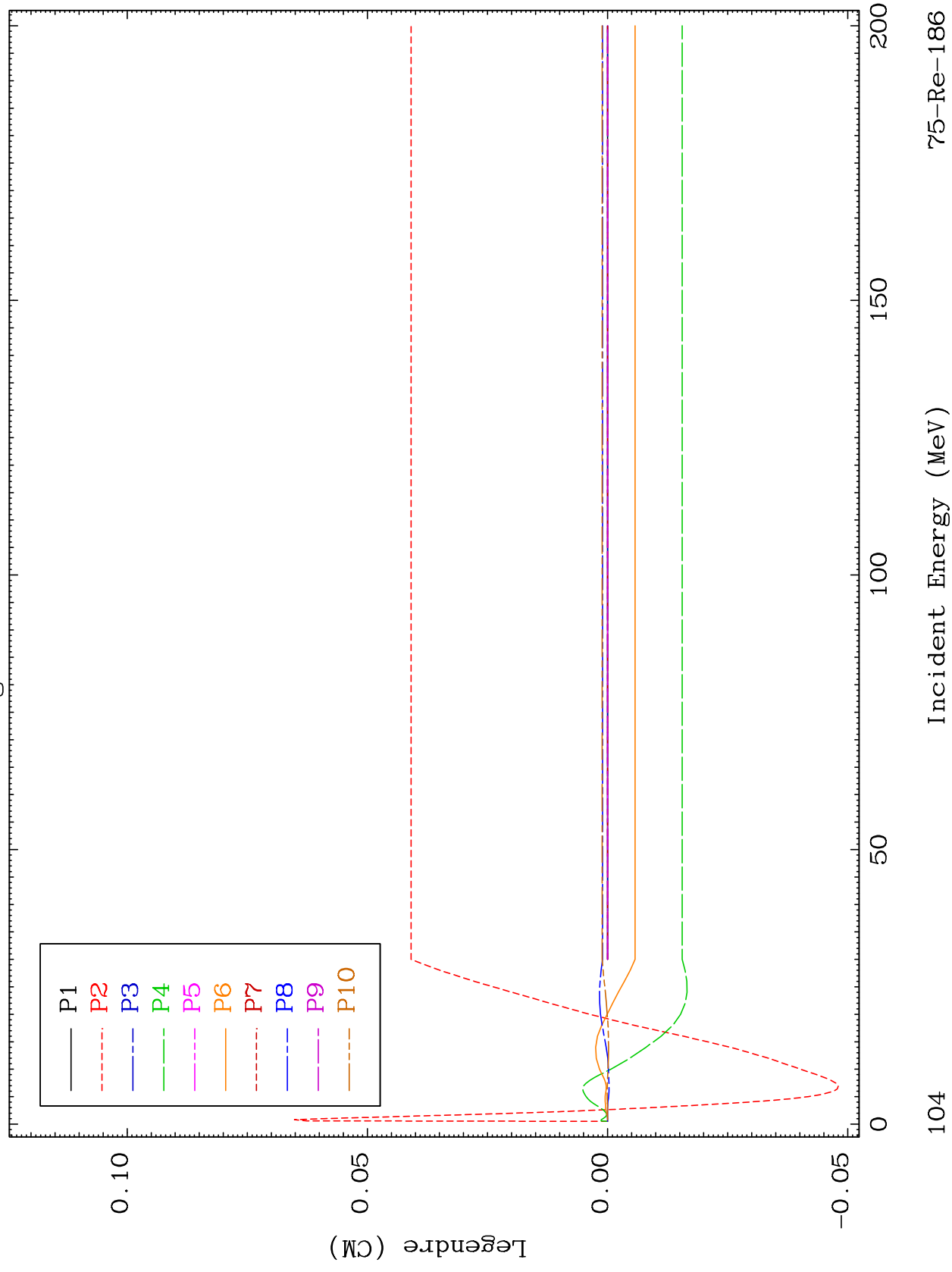
75-Re-186

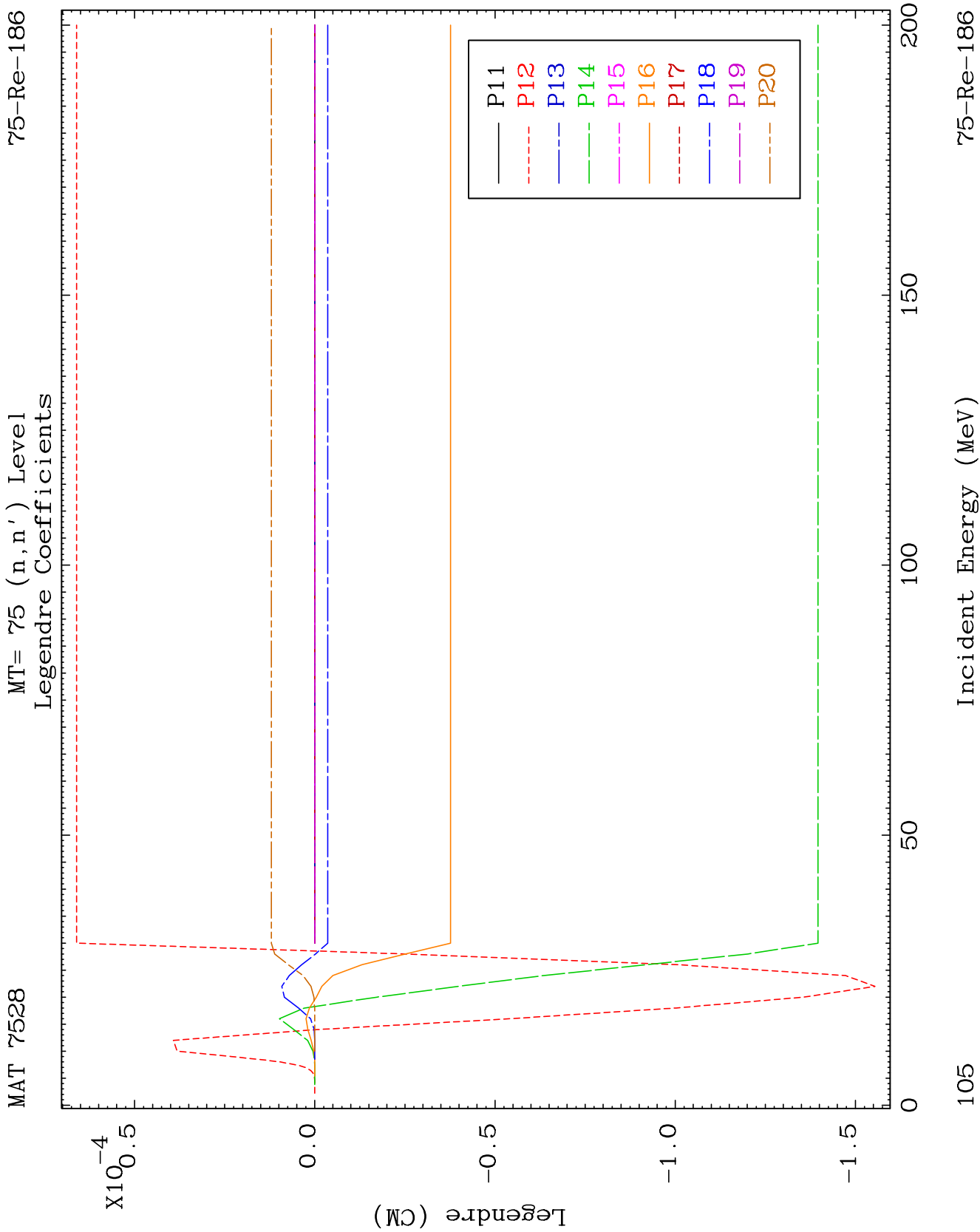


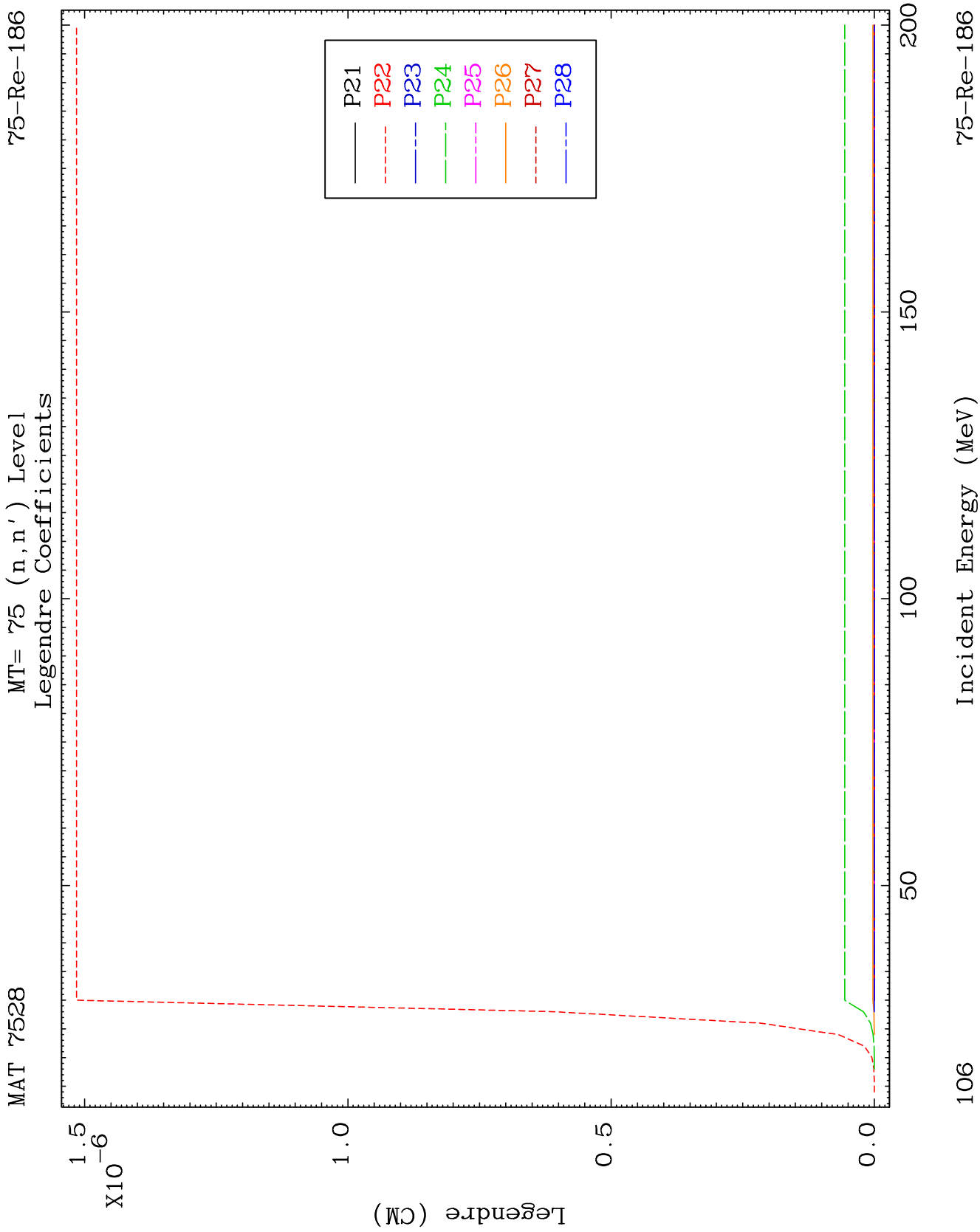
MAT 7528

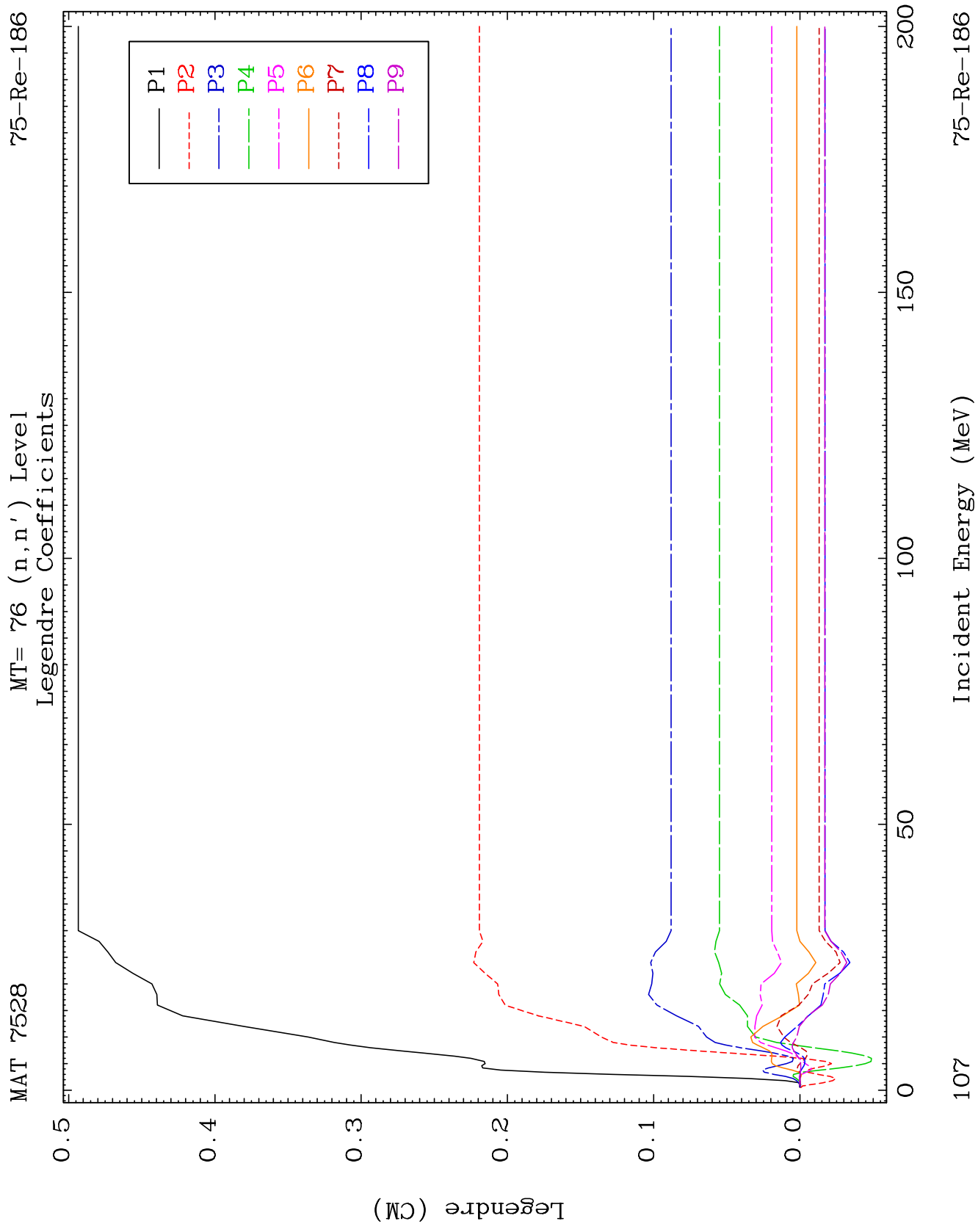
MT= 75 (n, n') Level
Legendre Coefficients

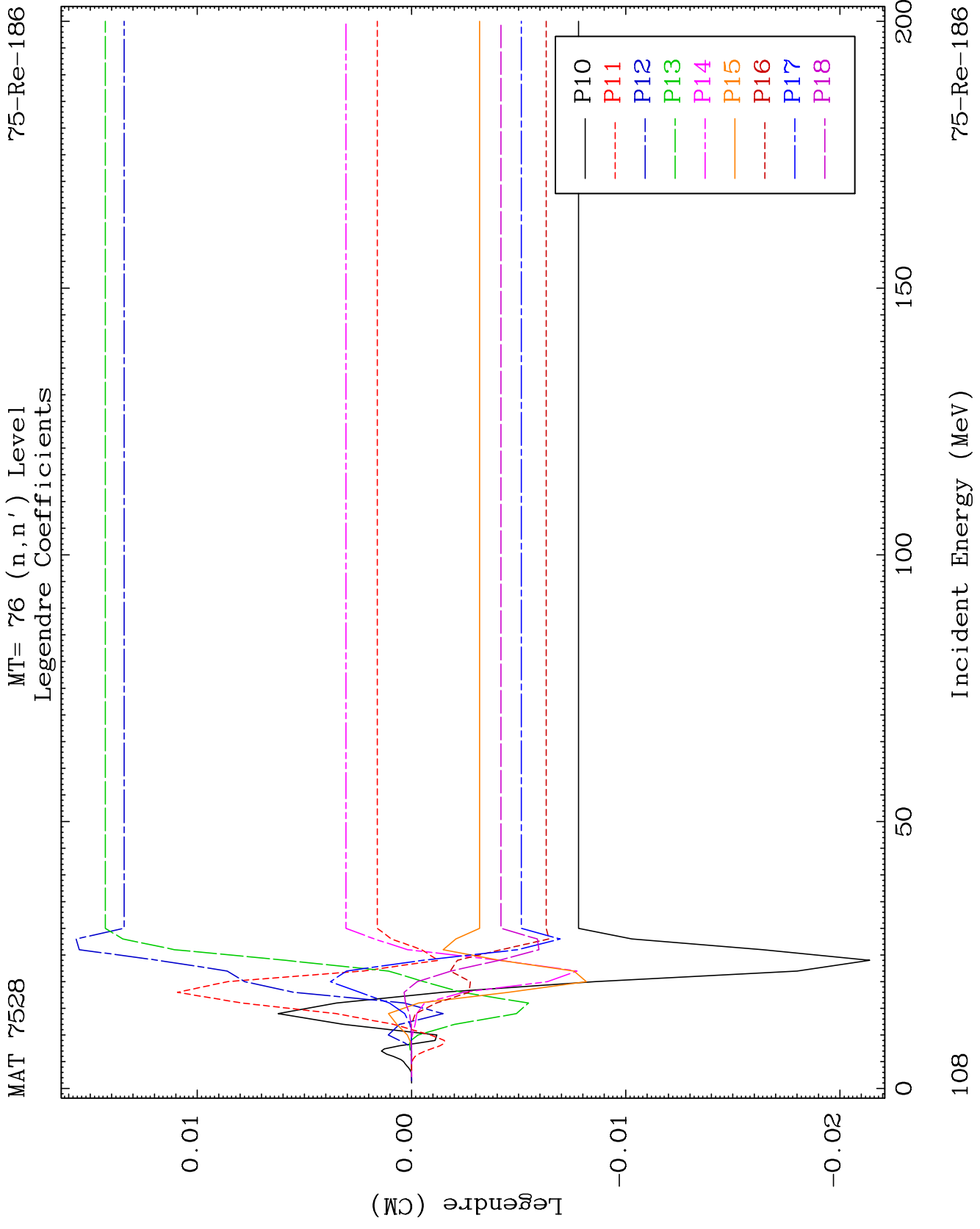
75-Re-186

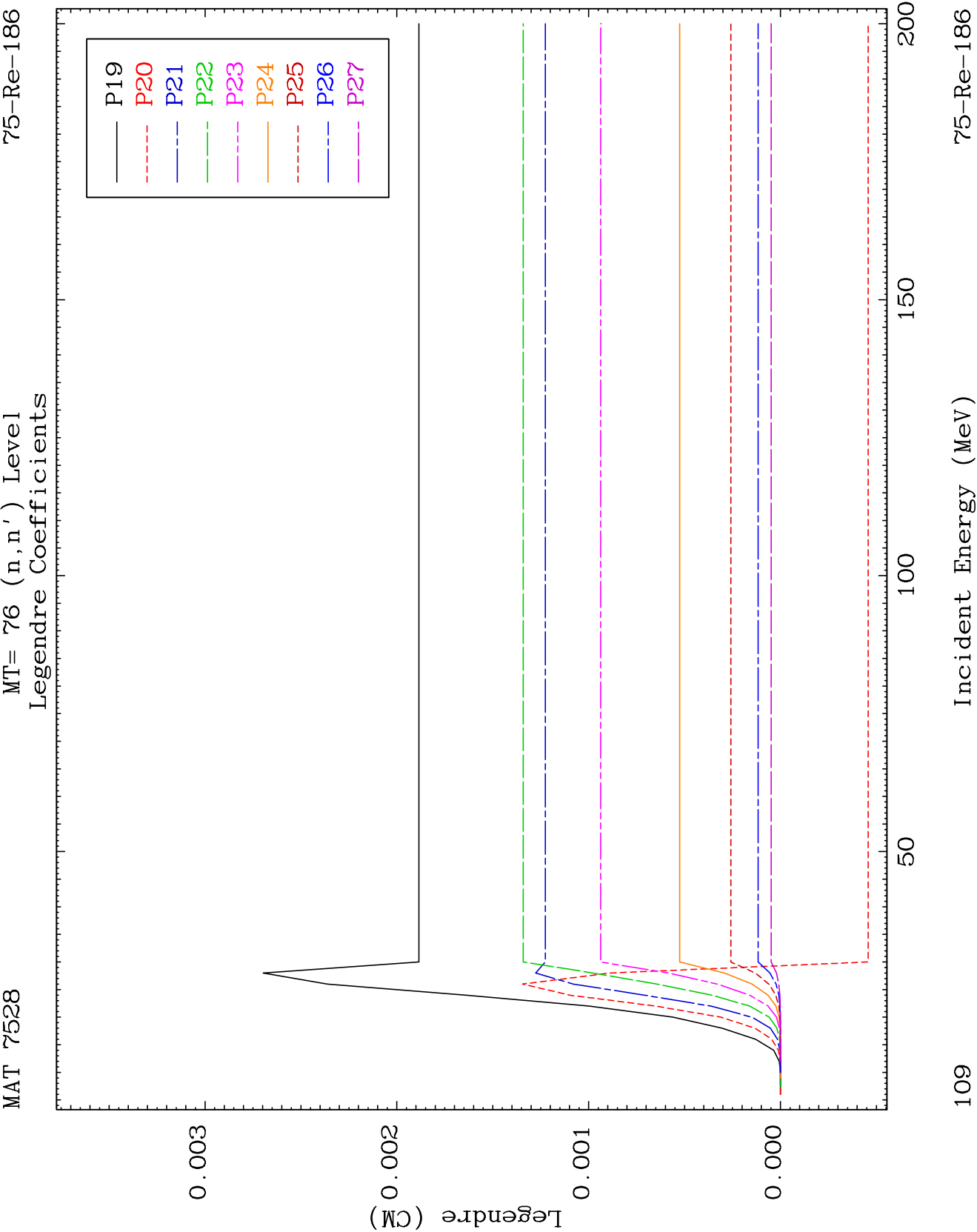


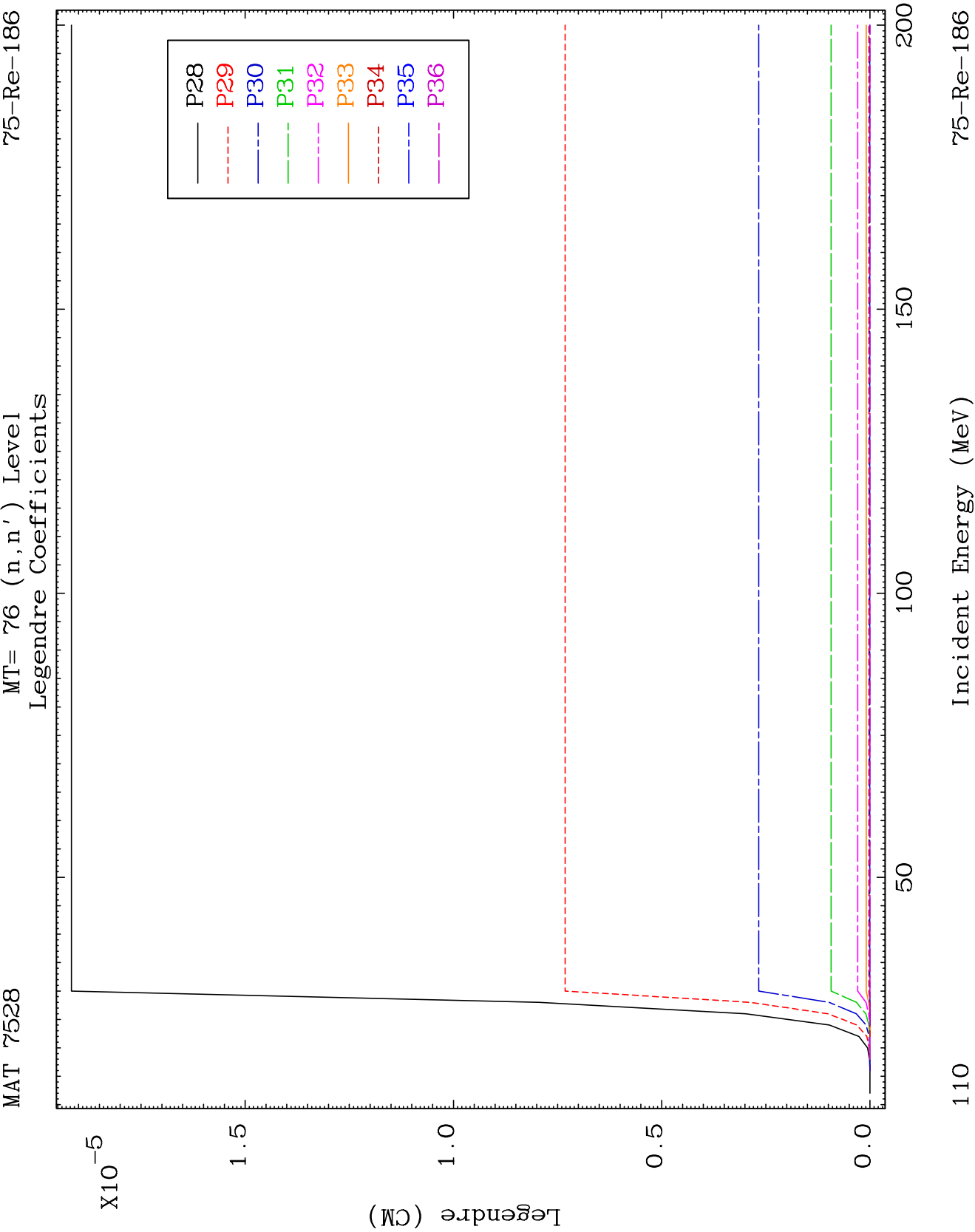




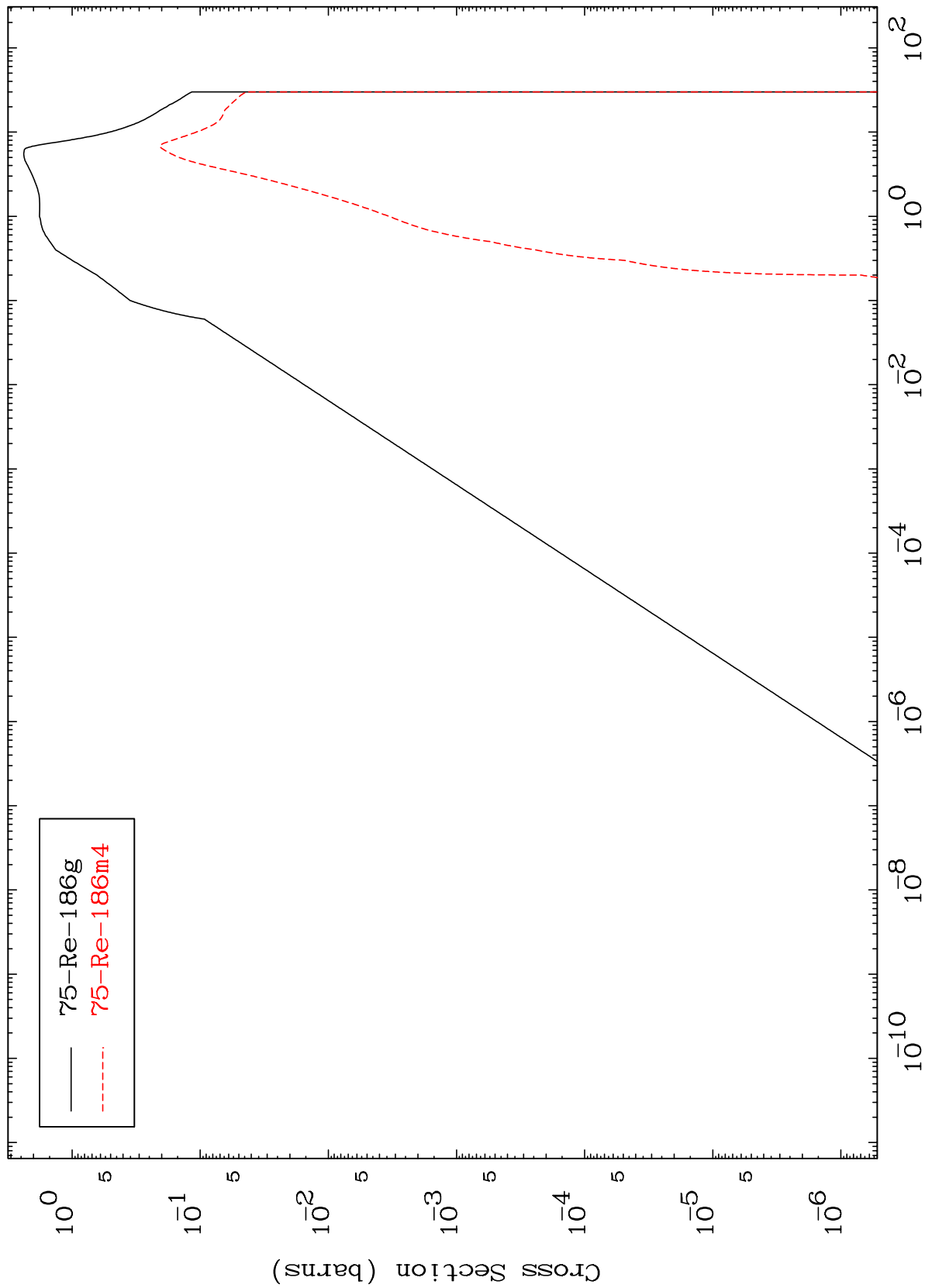




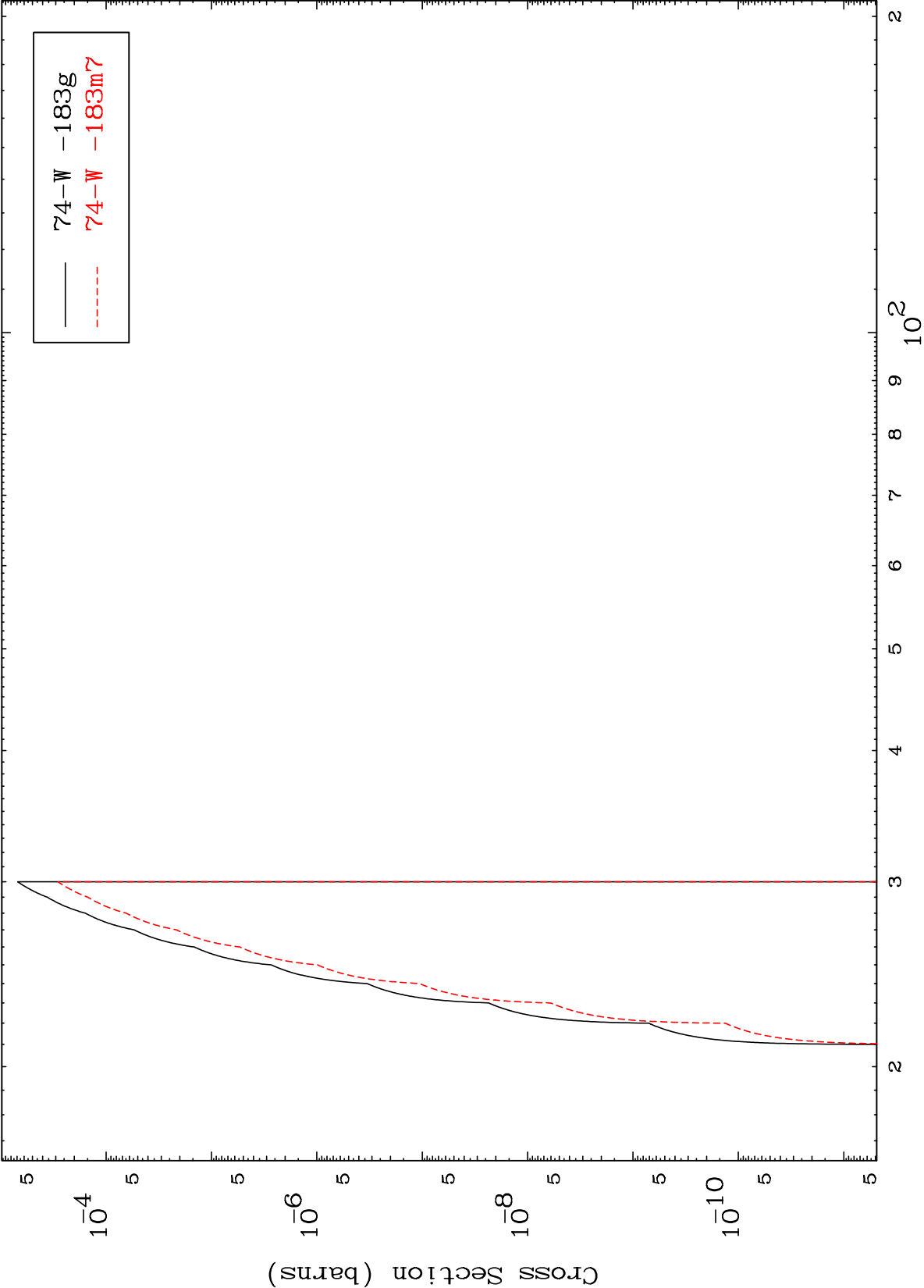


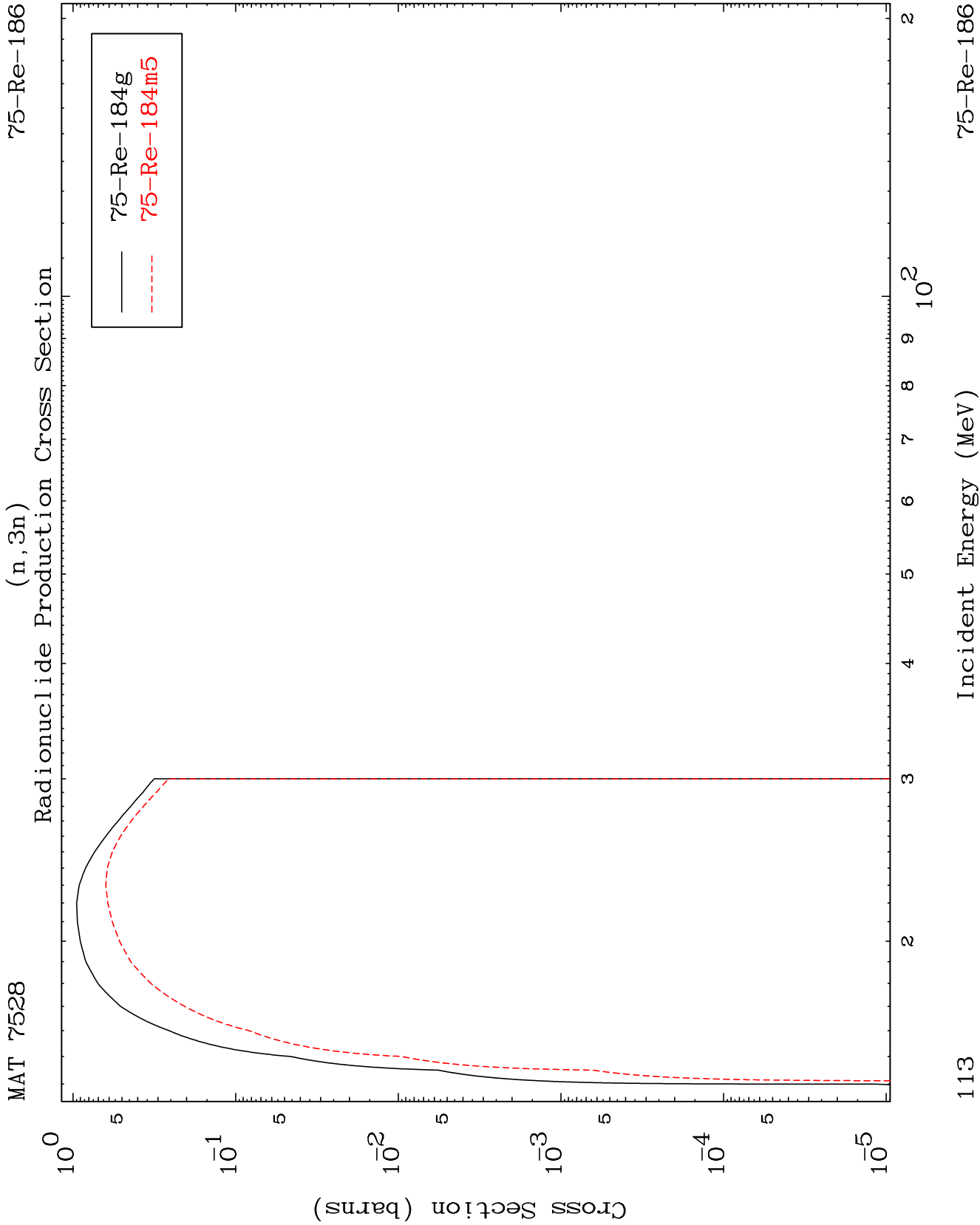


Radionuclide Production Cross Section



Radionuclide Production Cross Section



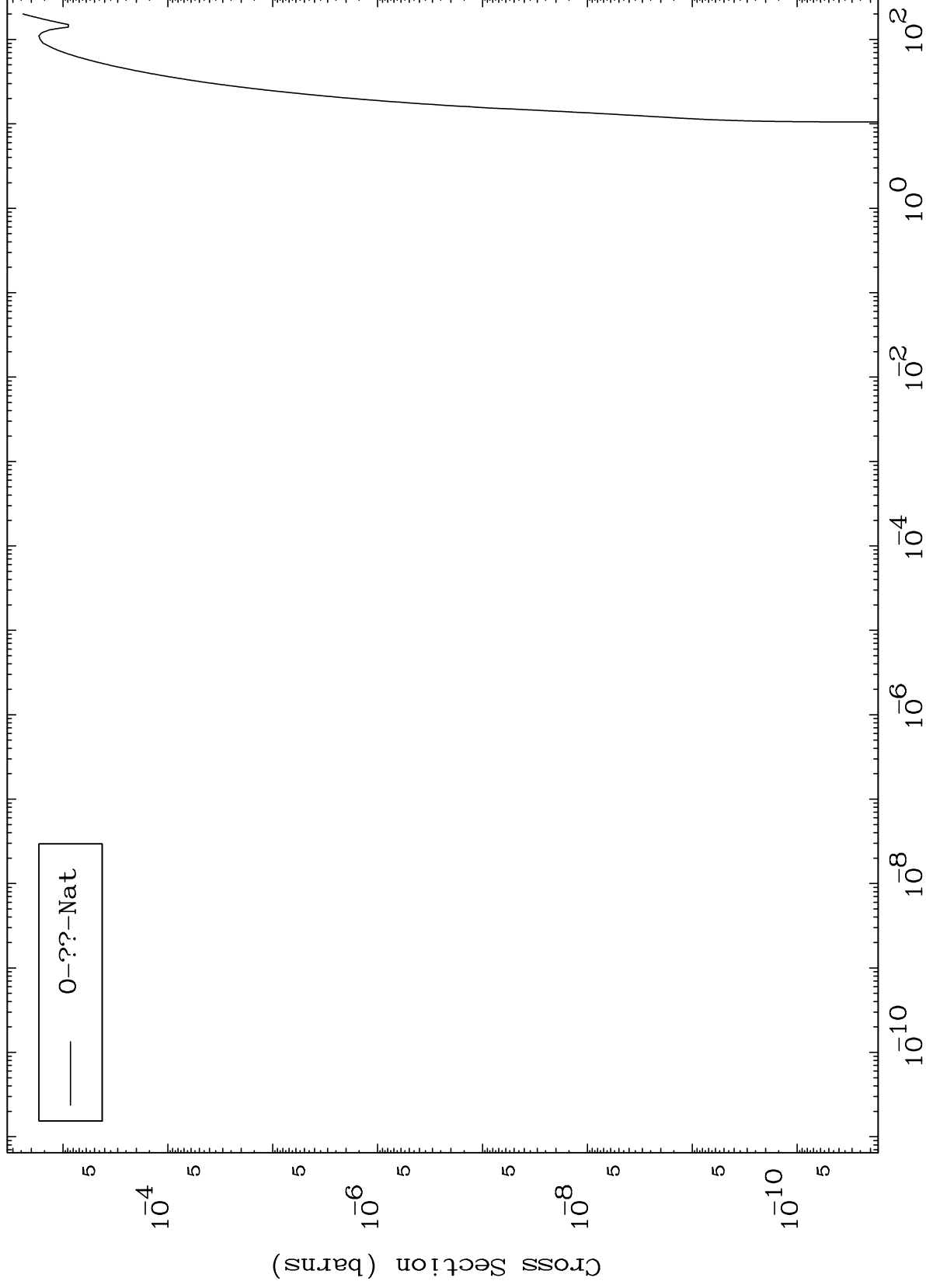


MAT 7528

Fission

⁷⁵Re-¹⁸⁶

Radionuclide Production Cross Section



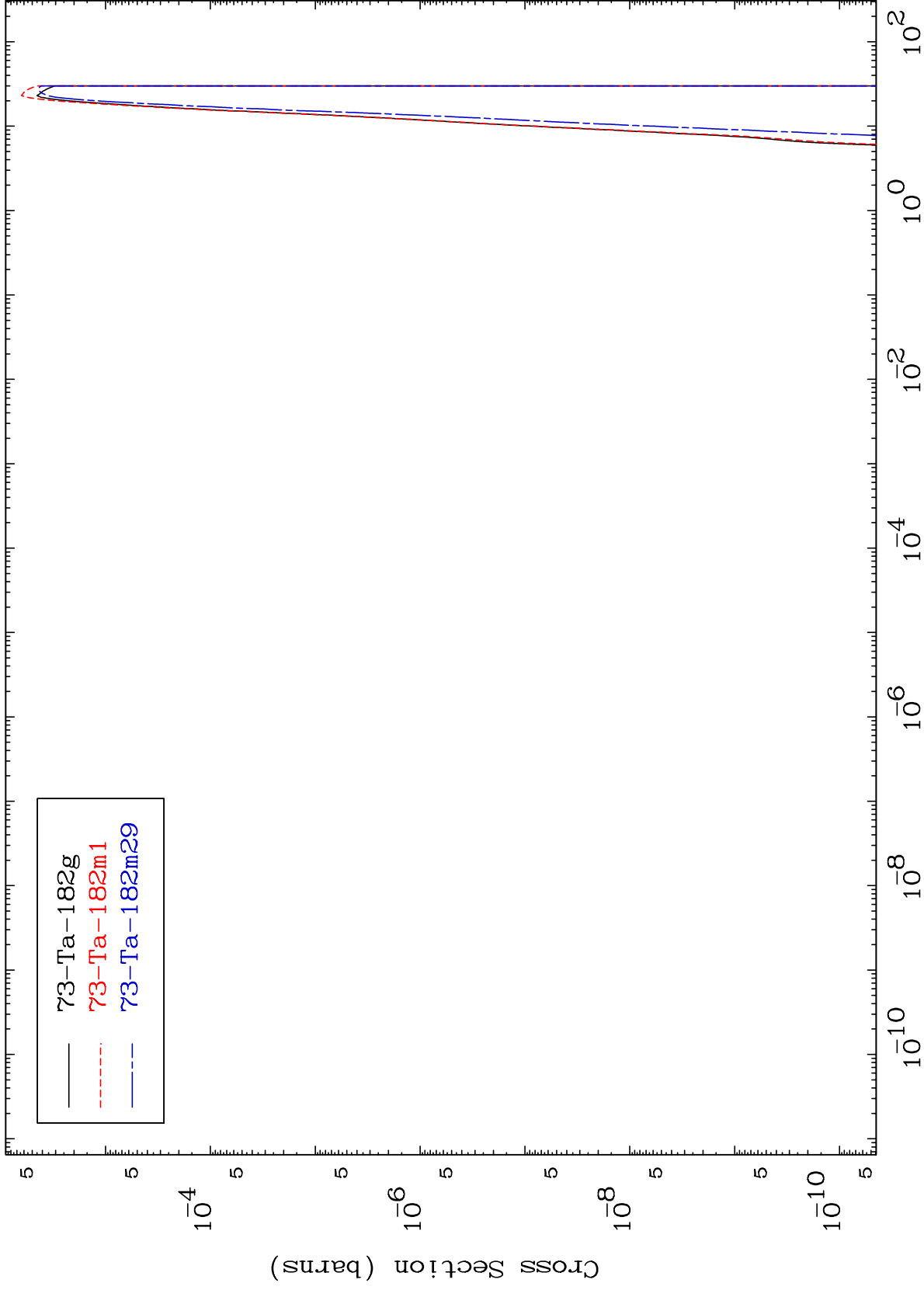
114

⁷⁵Re-¹⁸⁶

MAT 7528

75-Re-186

$(n,n') \alpha$
Radionuclide Production Cross Section

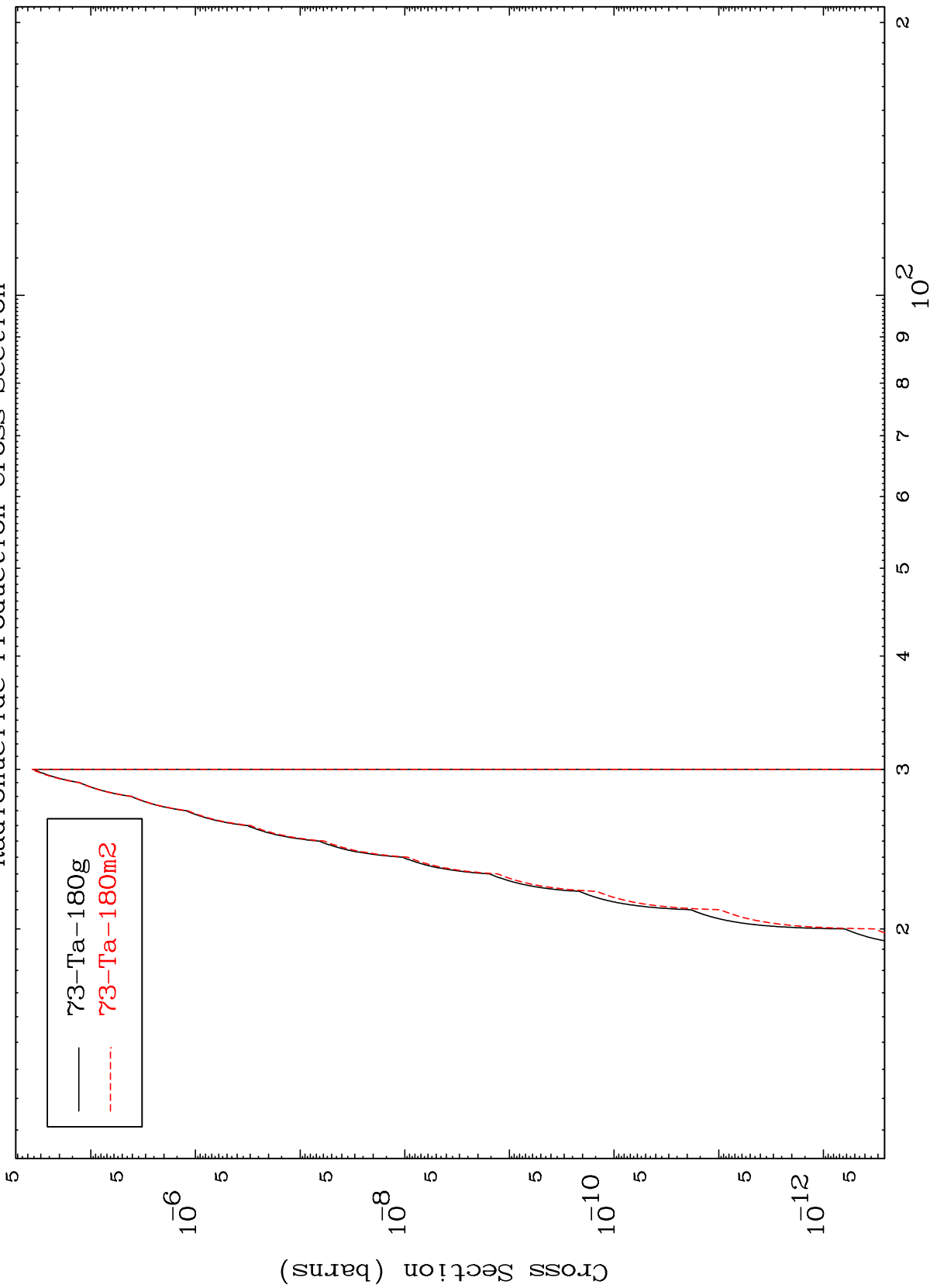


115

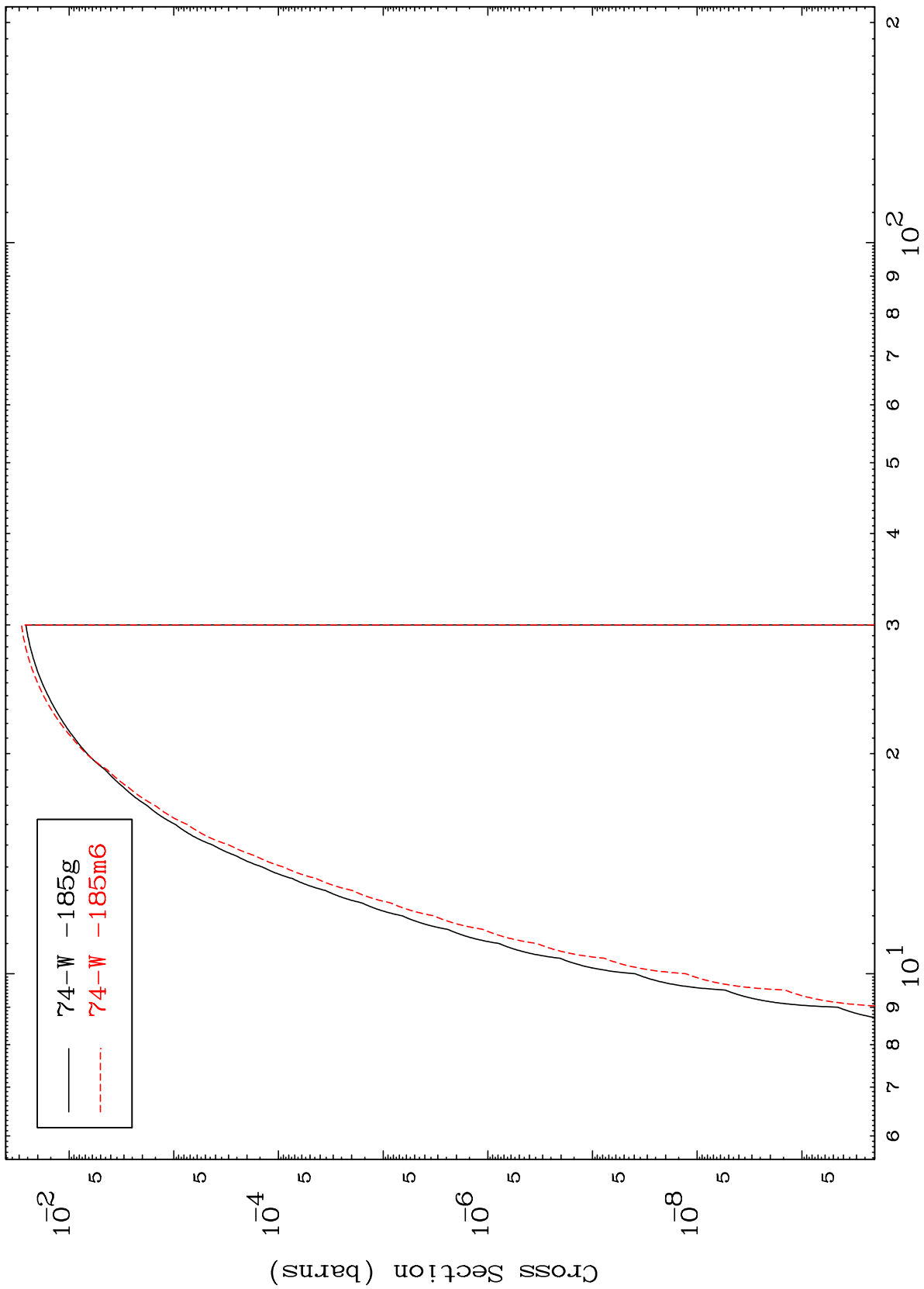
Incident Energy (MeV)

75-Re-186

Radionuclide Production Cross Section

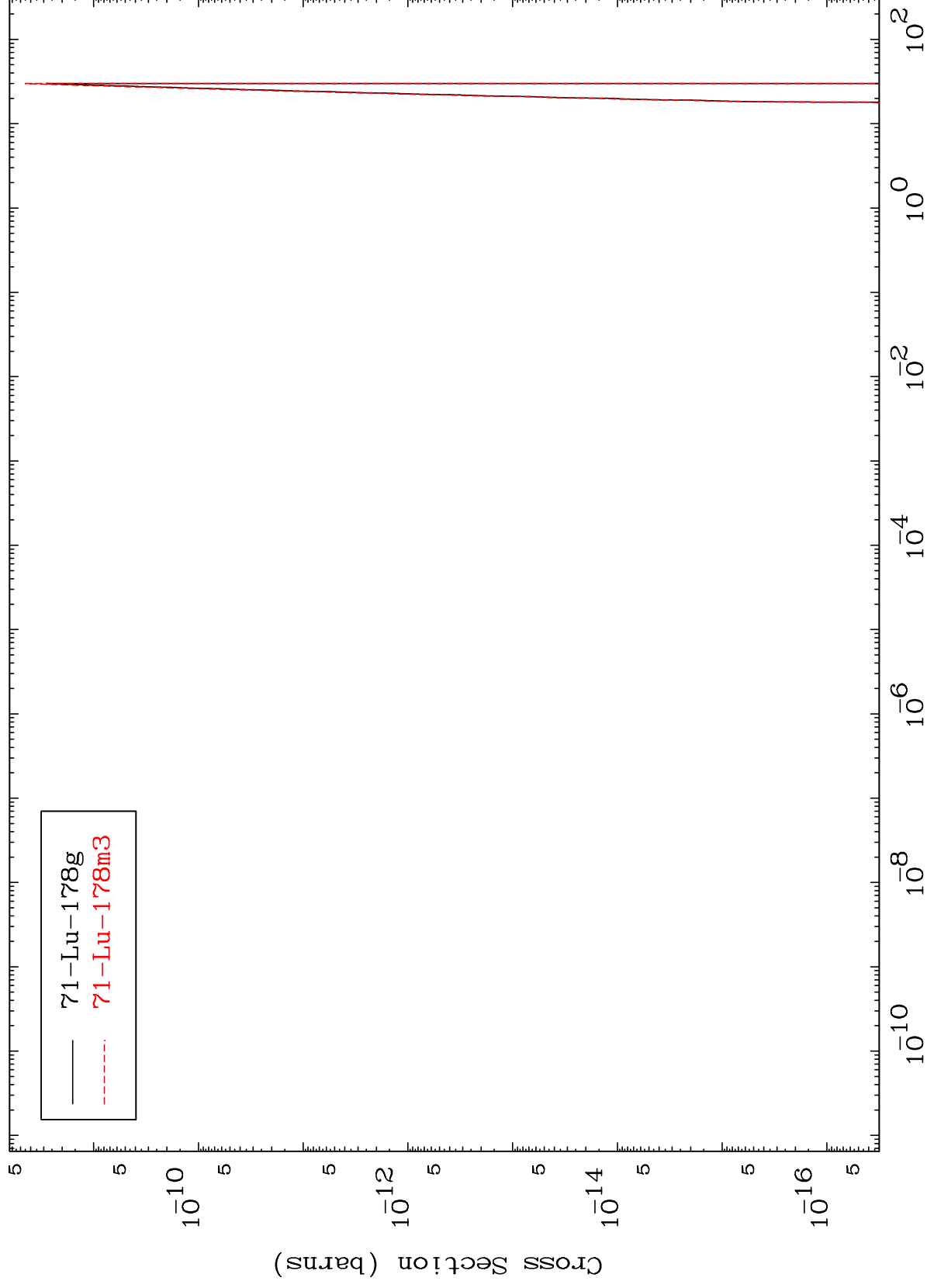


(n,n') p
Radionuclide Production Cross Section

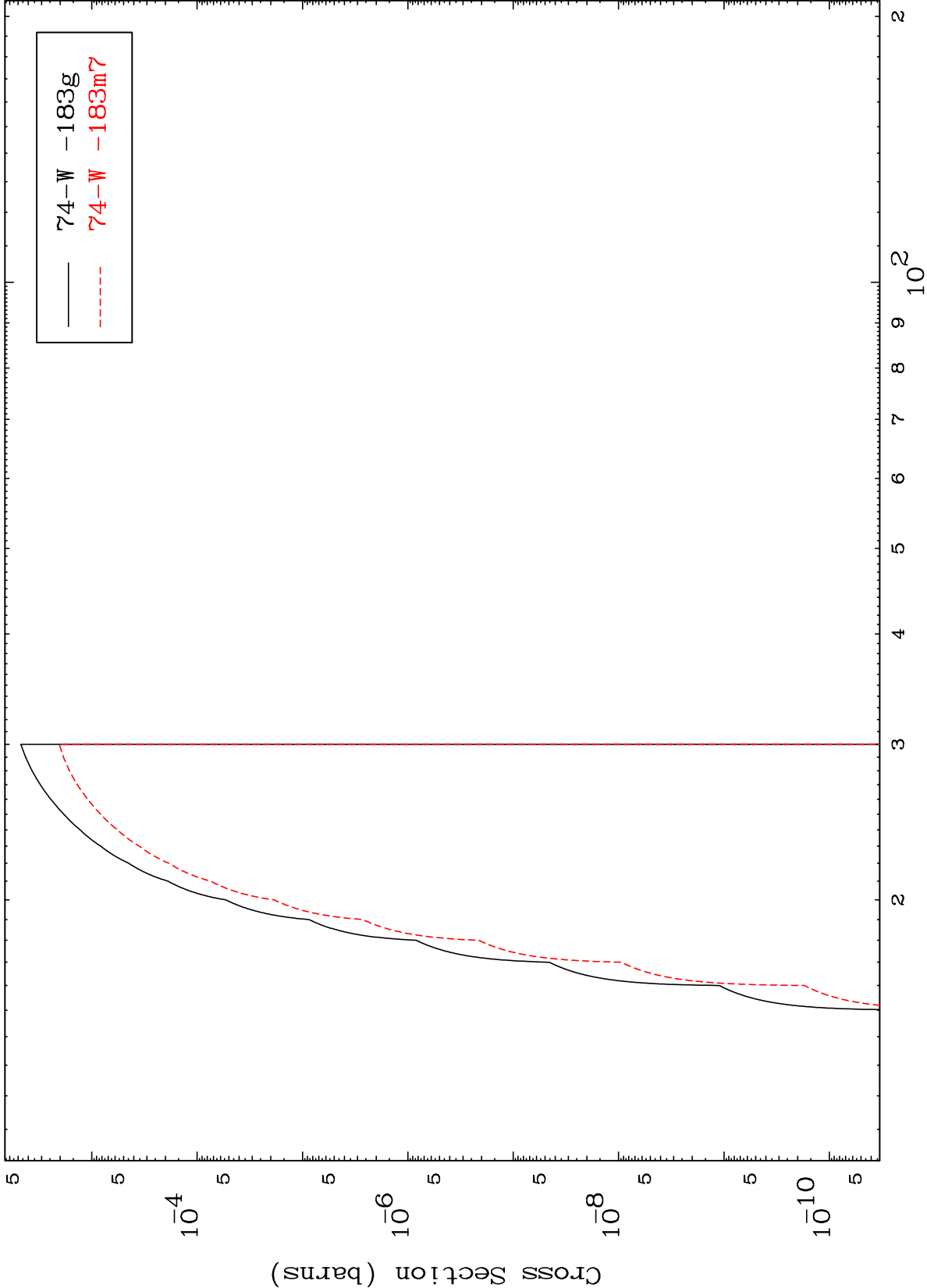


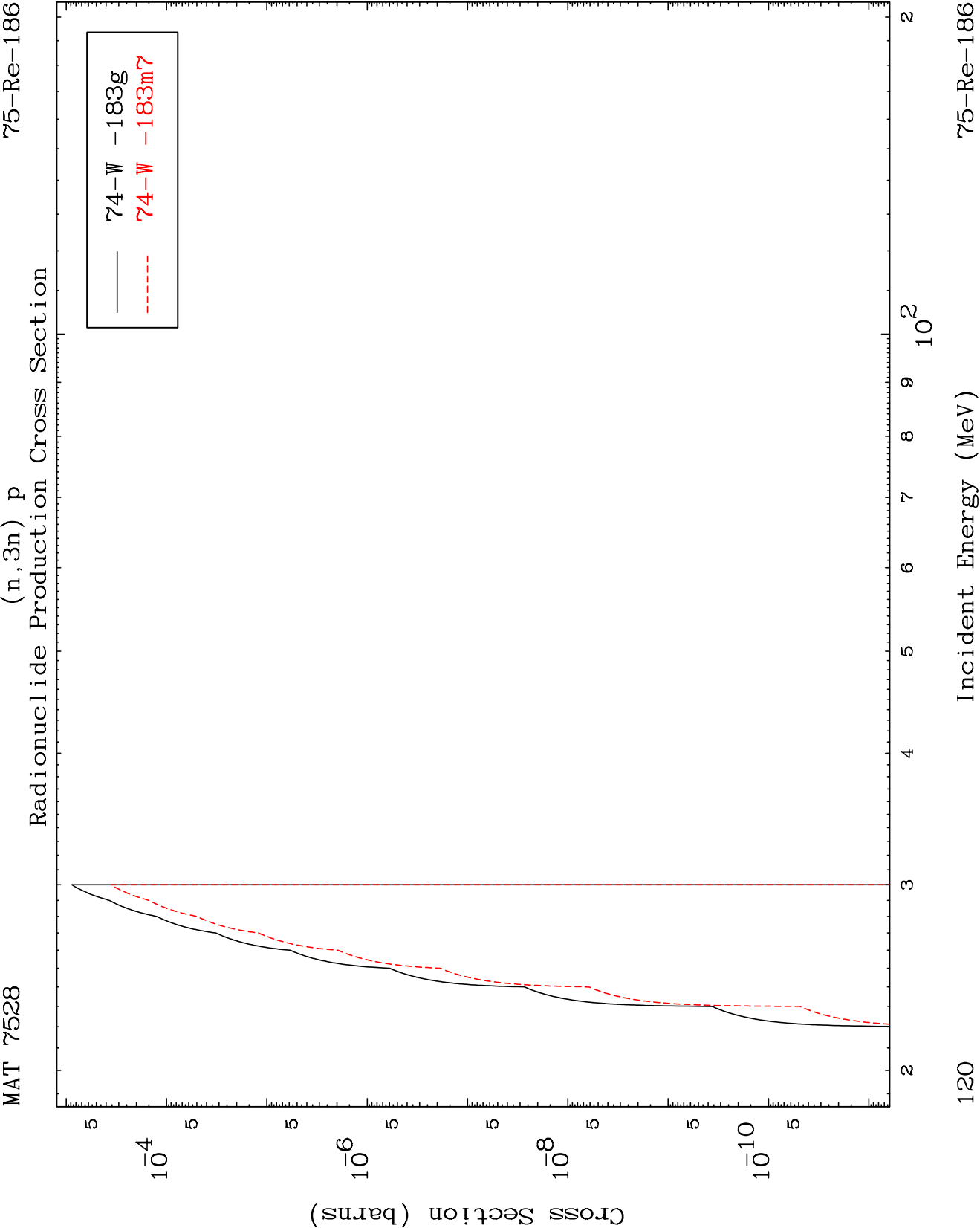
Incident Energy (MeV)

Radionuclide Production Cross Section



Radionuclide Production Cross Section

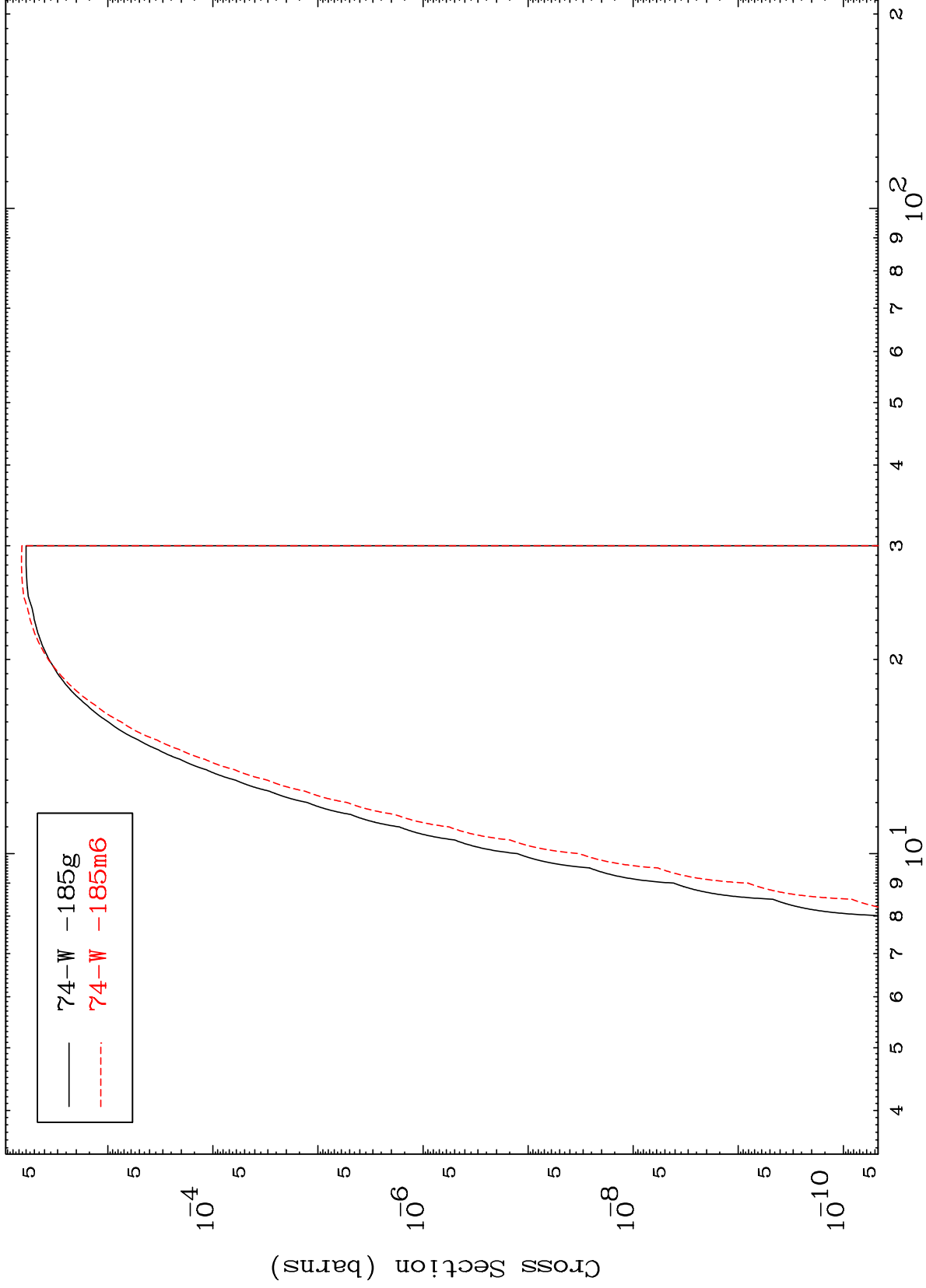




MAT 7528

75-Re-186

Radionuclide Production Cross Section



121

Incident Energy (MeV)

75-Re-186

Radionuclide Production Cross Section

