

Program EVALPLOT
(Version 2021-1)

by

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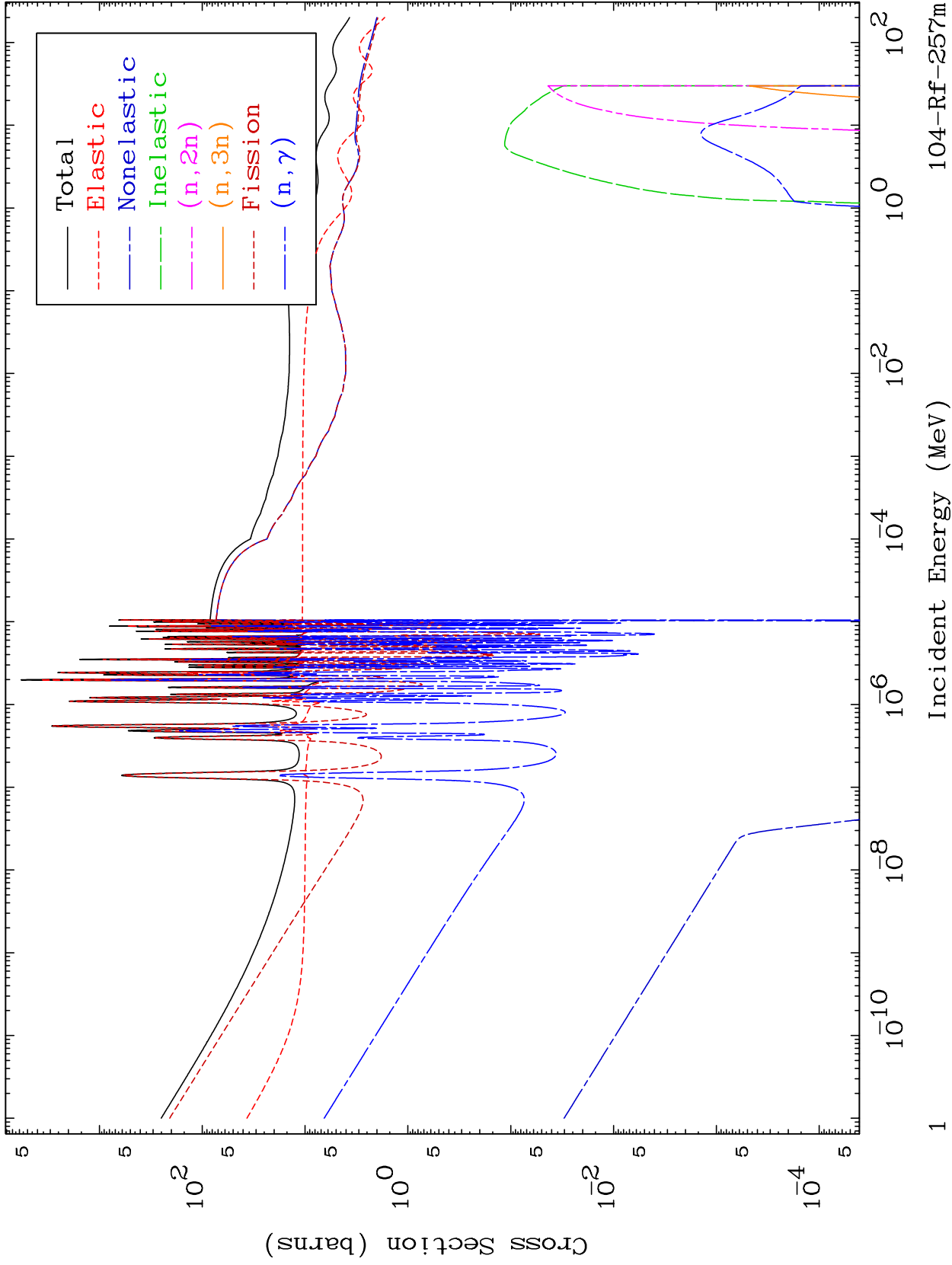
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 457

Neutron Major
293 Kelvin Cross Sections

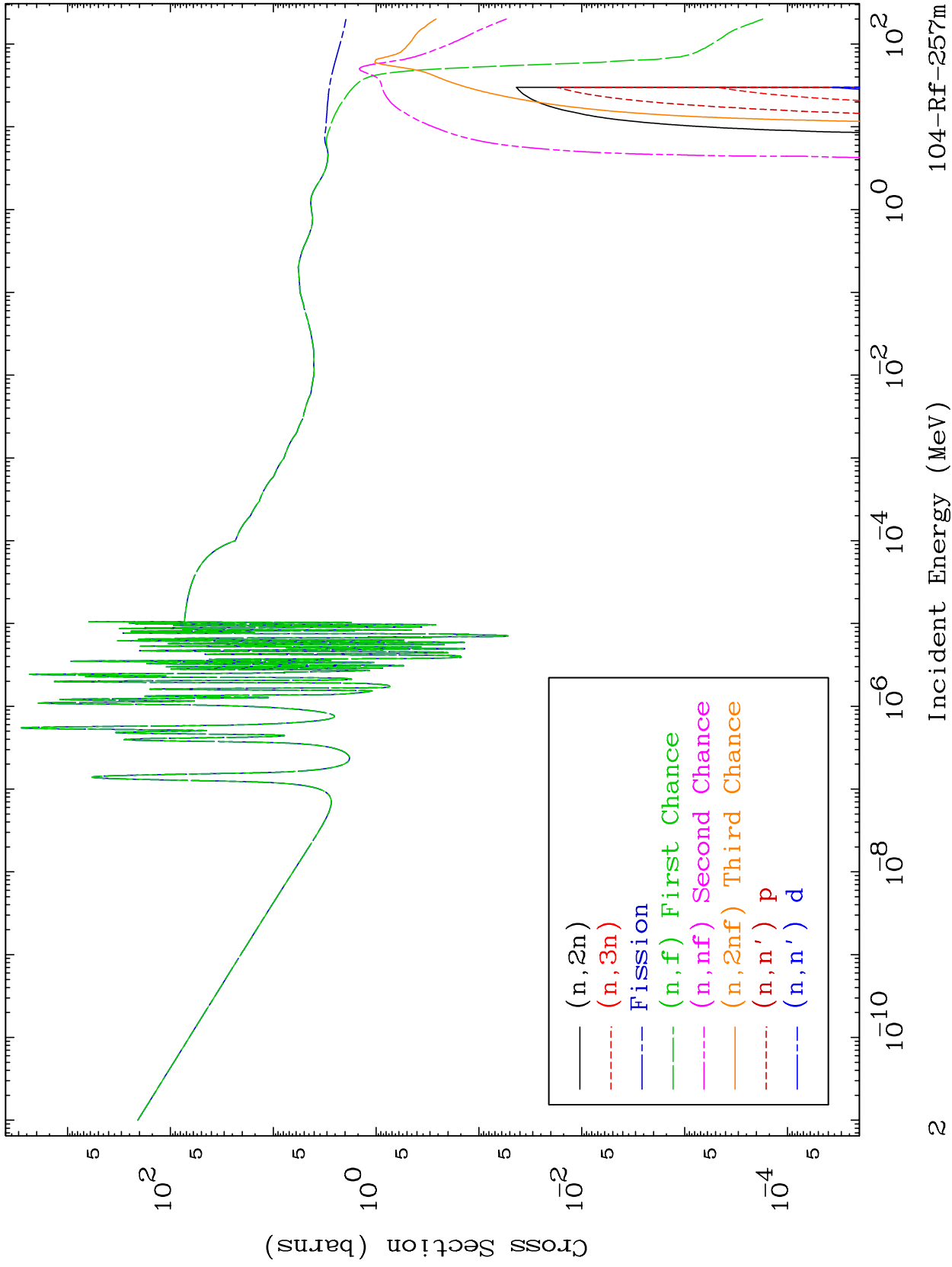
104-Rf-257m



MAT 457

Neutron Absorption
293 Kelvin Cross Sections

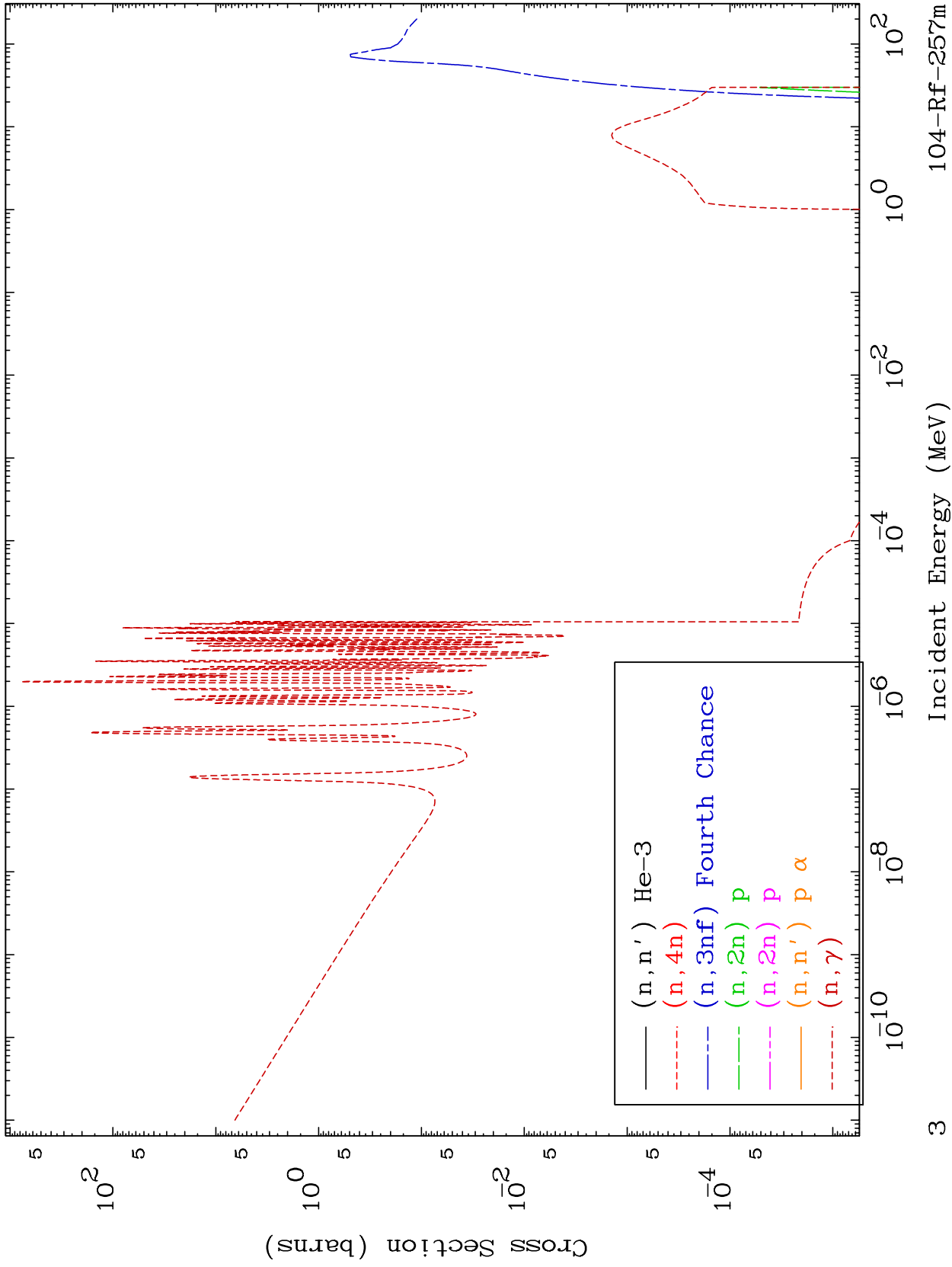
104-Rf-257m



MAT 457

Neutron Absorption
293 Kelvin Cross Sections

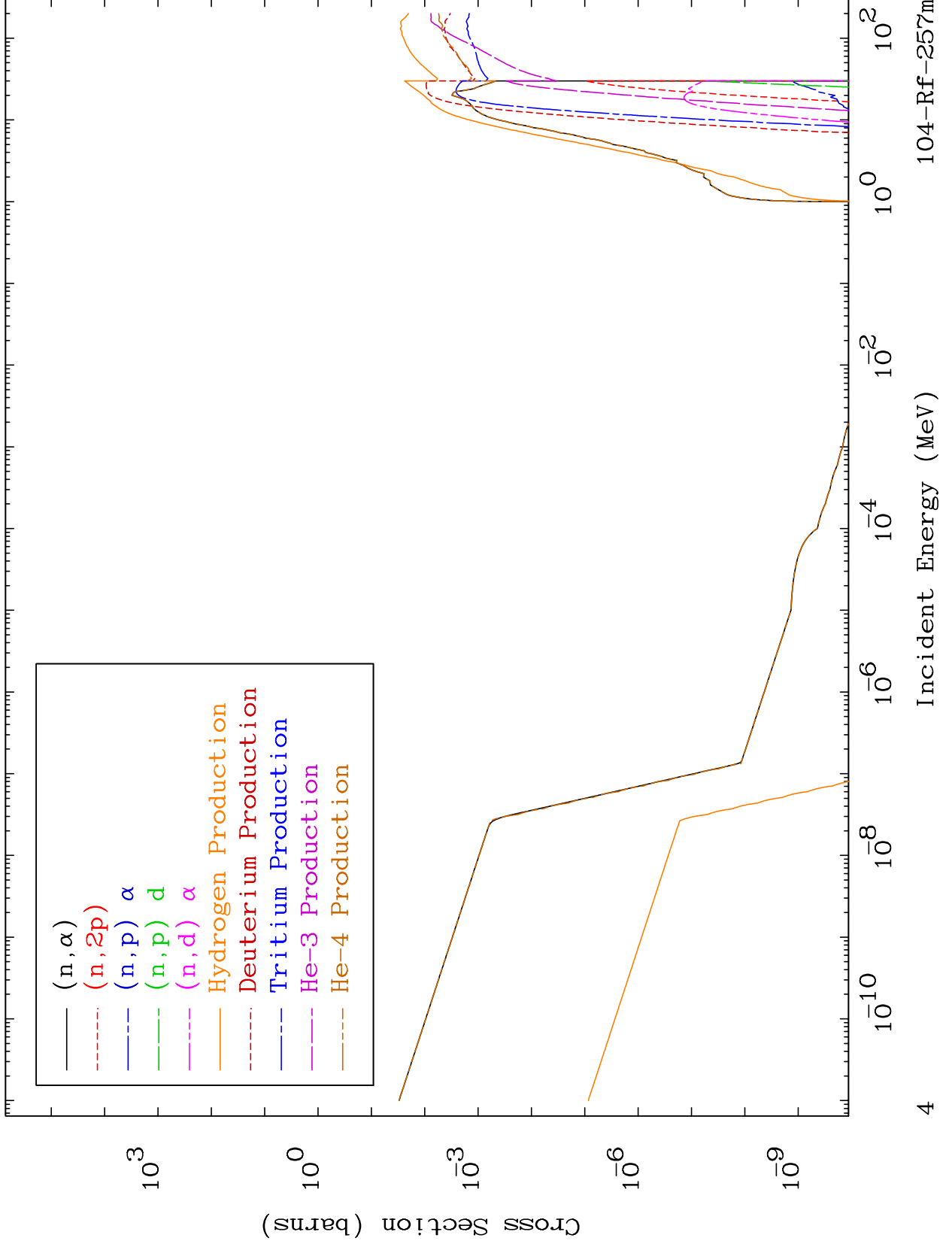
104-Rf-257m



MAT 457

Neutron Absorption
293 Kelvin Cross Sections

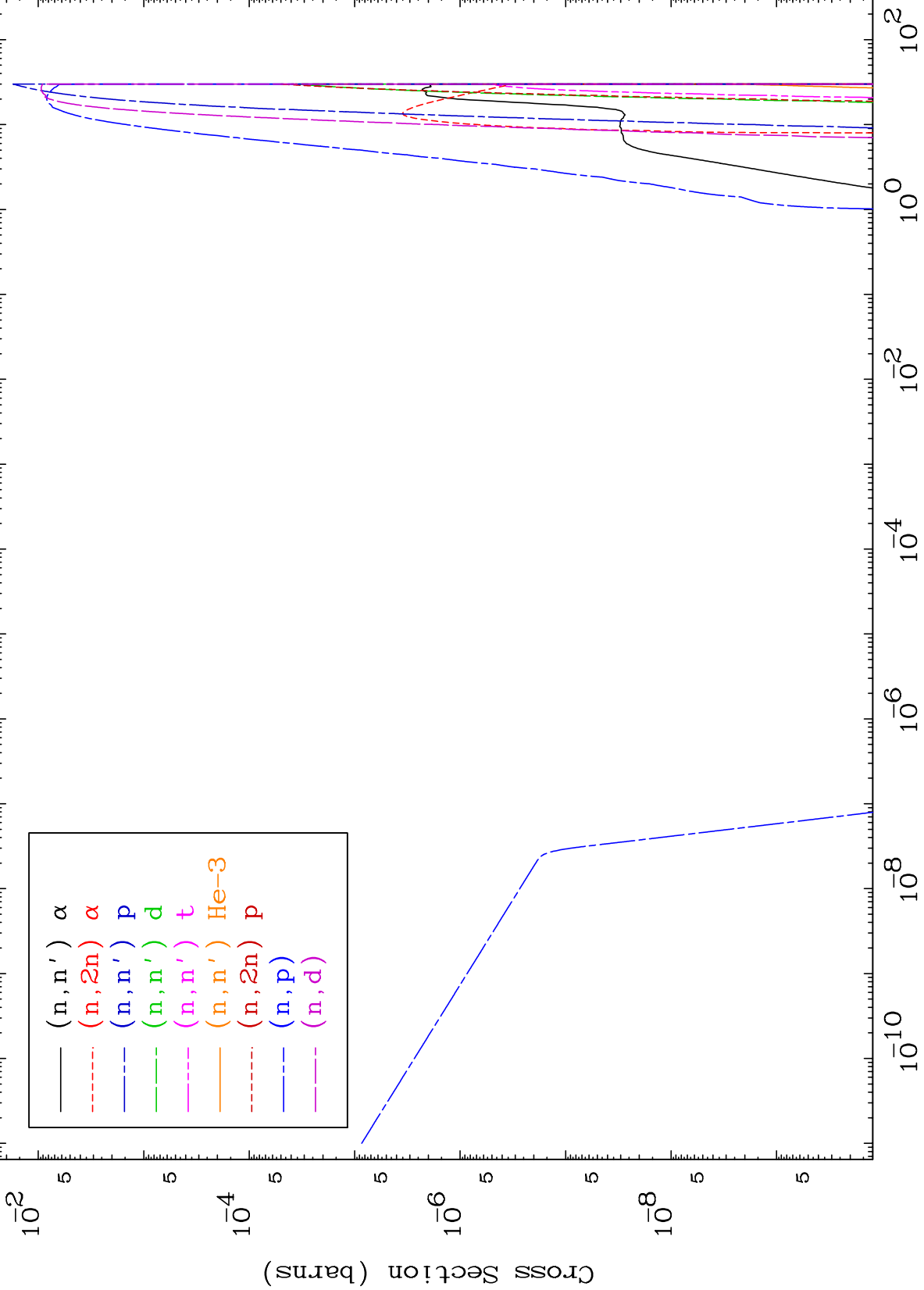
104-Rf-257m



MAT 457

Charged Particle
293 Kelvin Cross Sections

104-Rf-257m



5

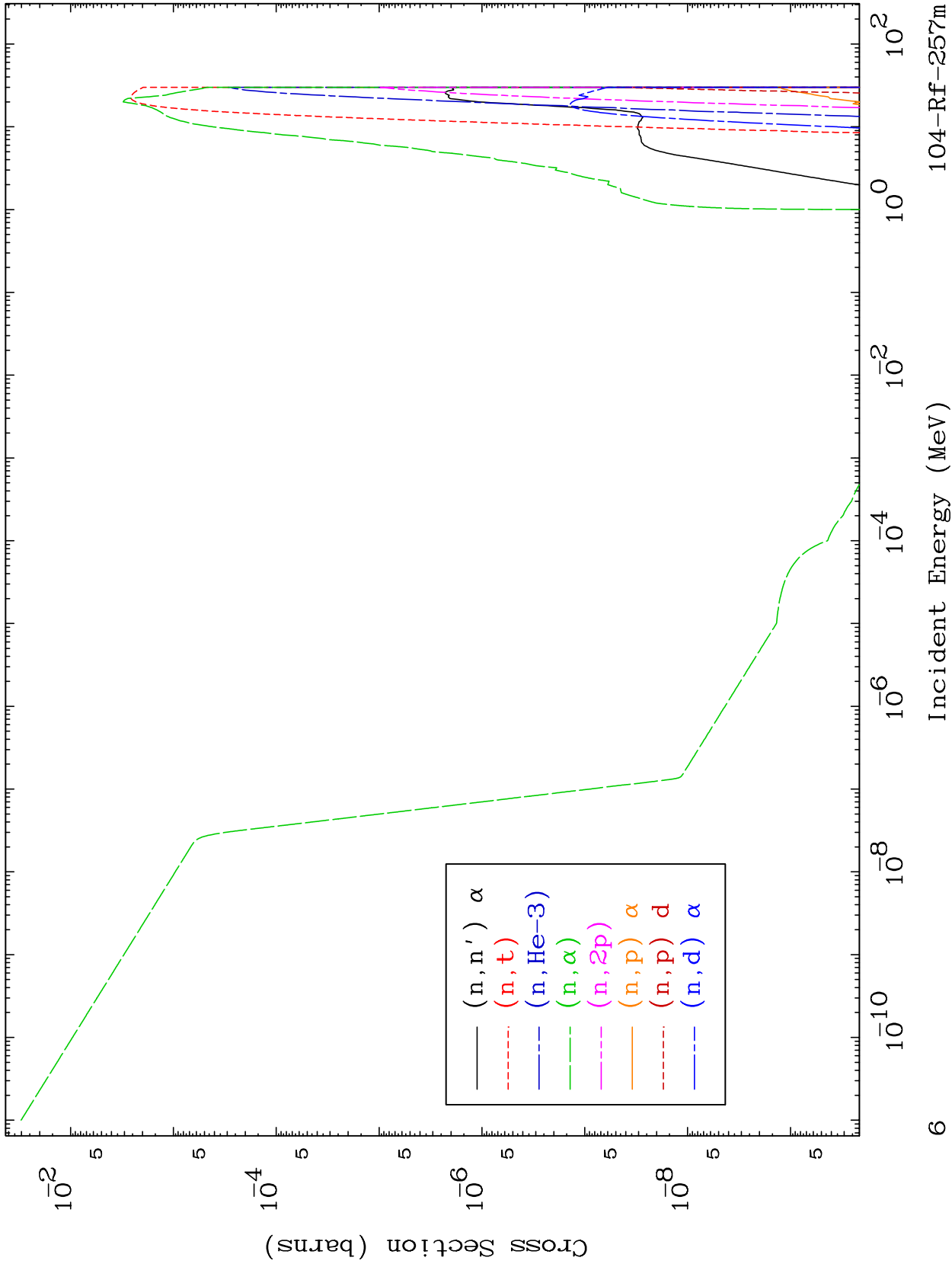
Incident Energy (MeV)

104-Rf-257m

MAT 457

Charged Particle
293 Kelvin Cross Sections

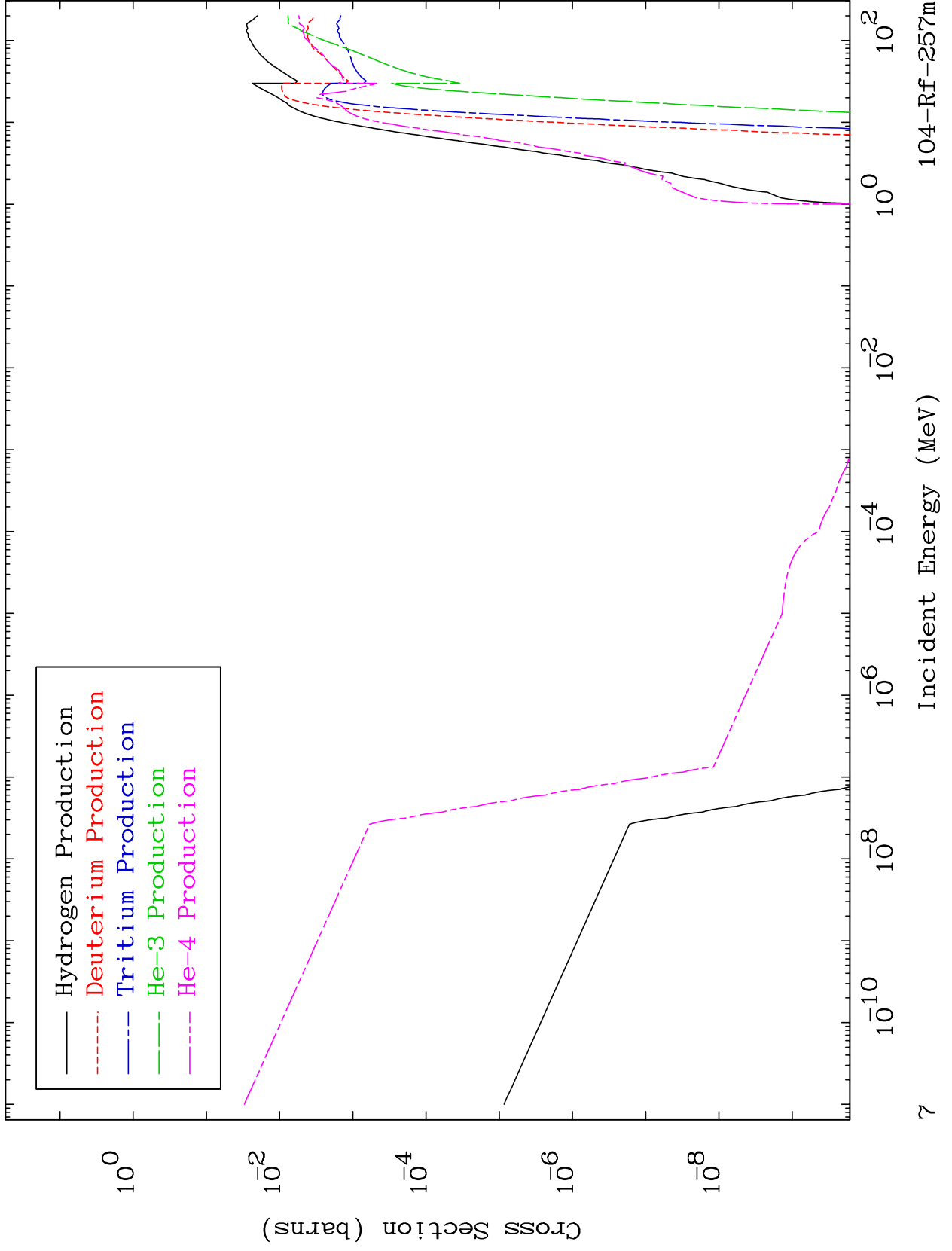
104-Rf-257m



MAT 457

Particle Production
293 Kelvin Cross Sections

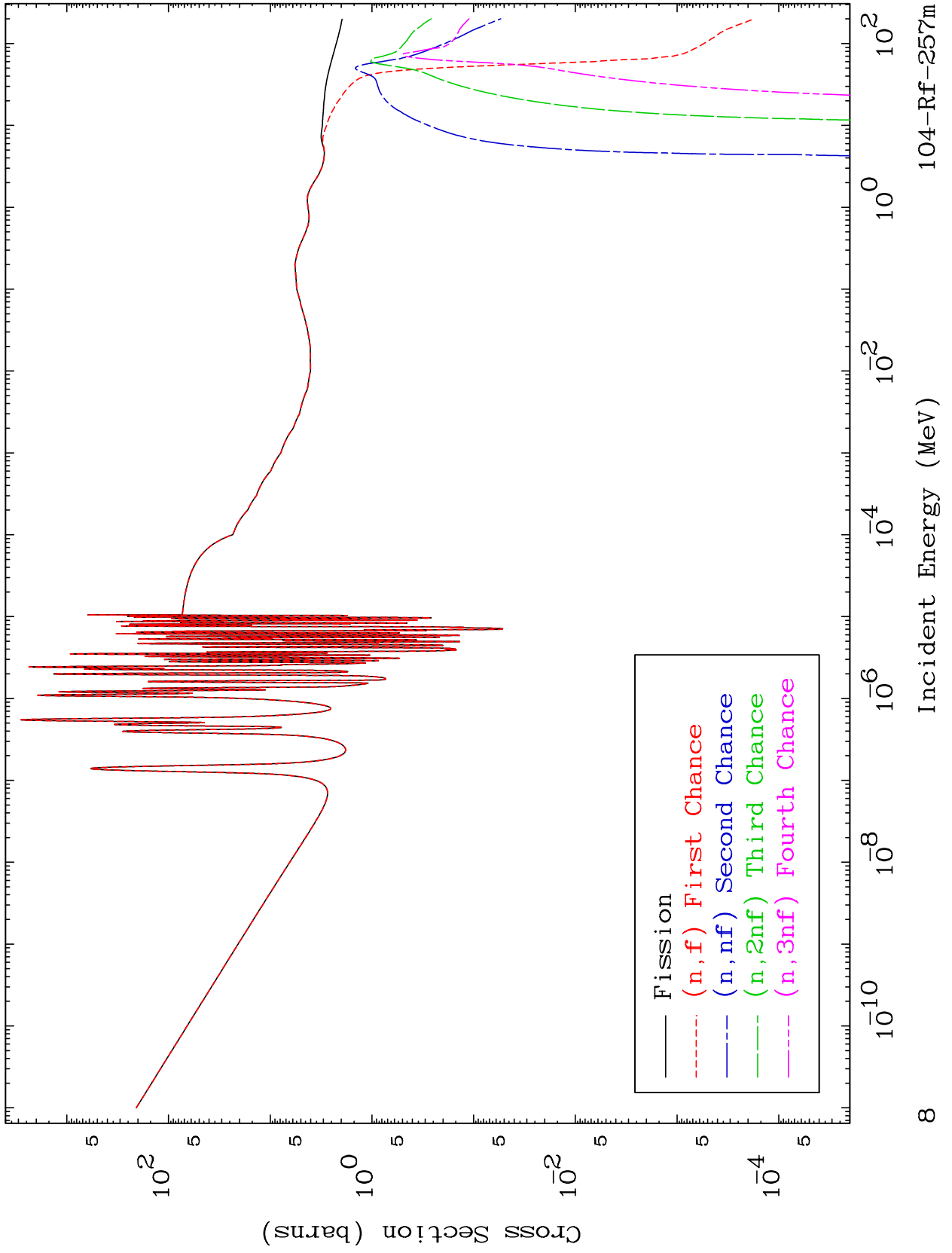
104-Rf-257m



MAT 457

Fission
293 Kelvin Cross Sections

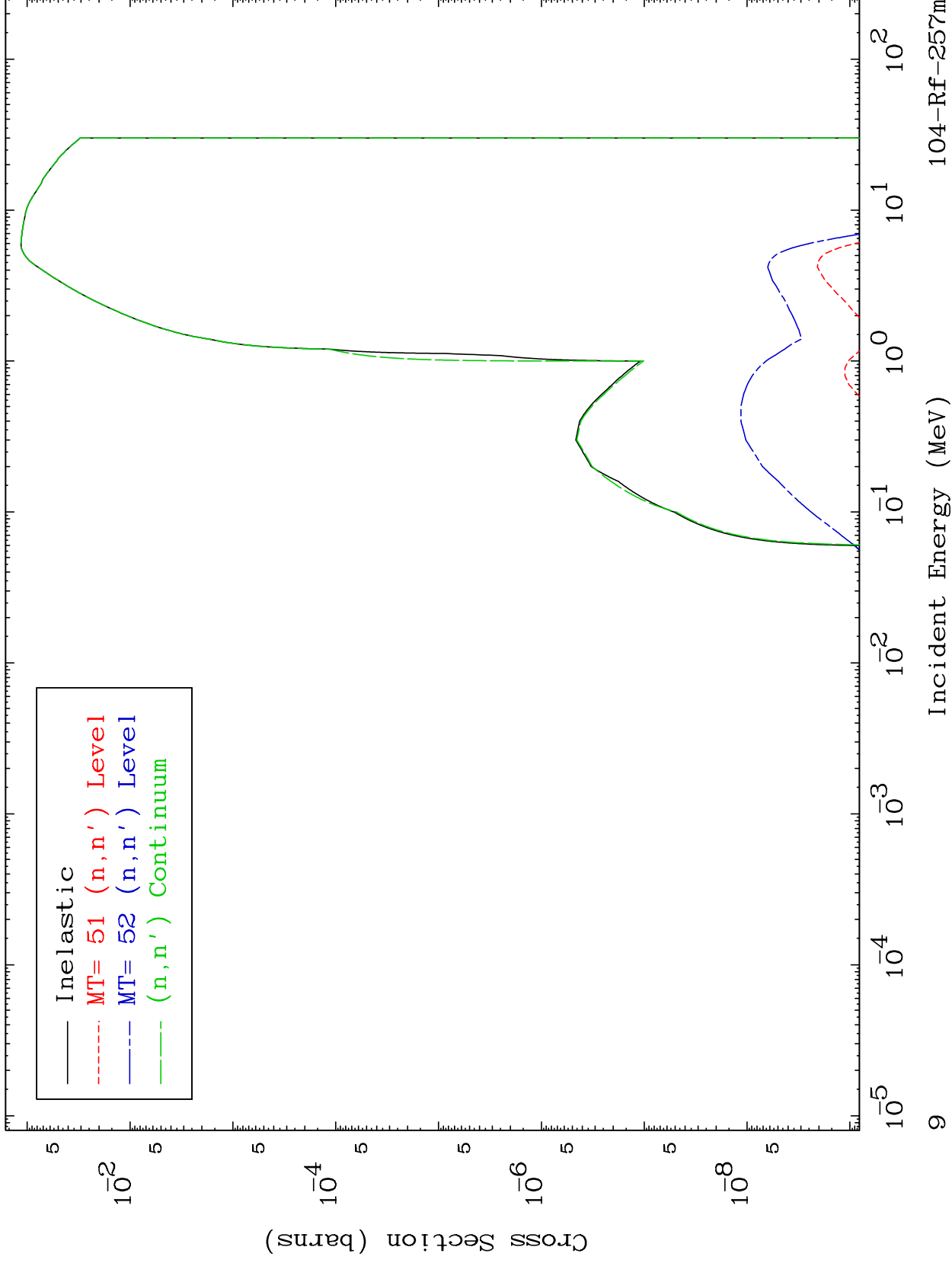
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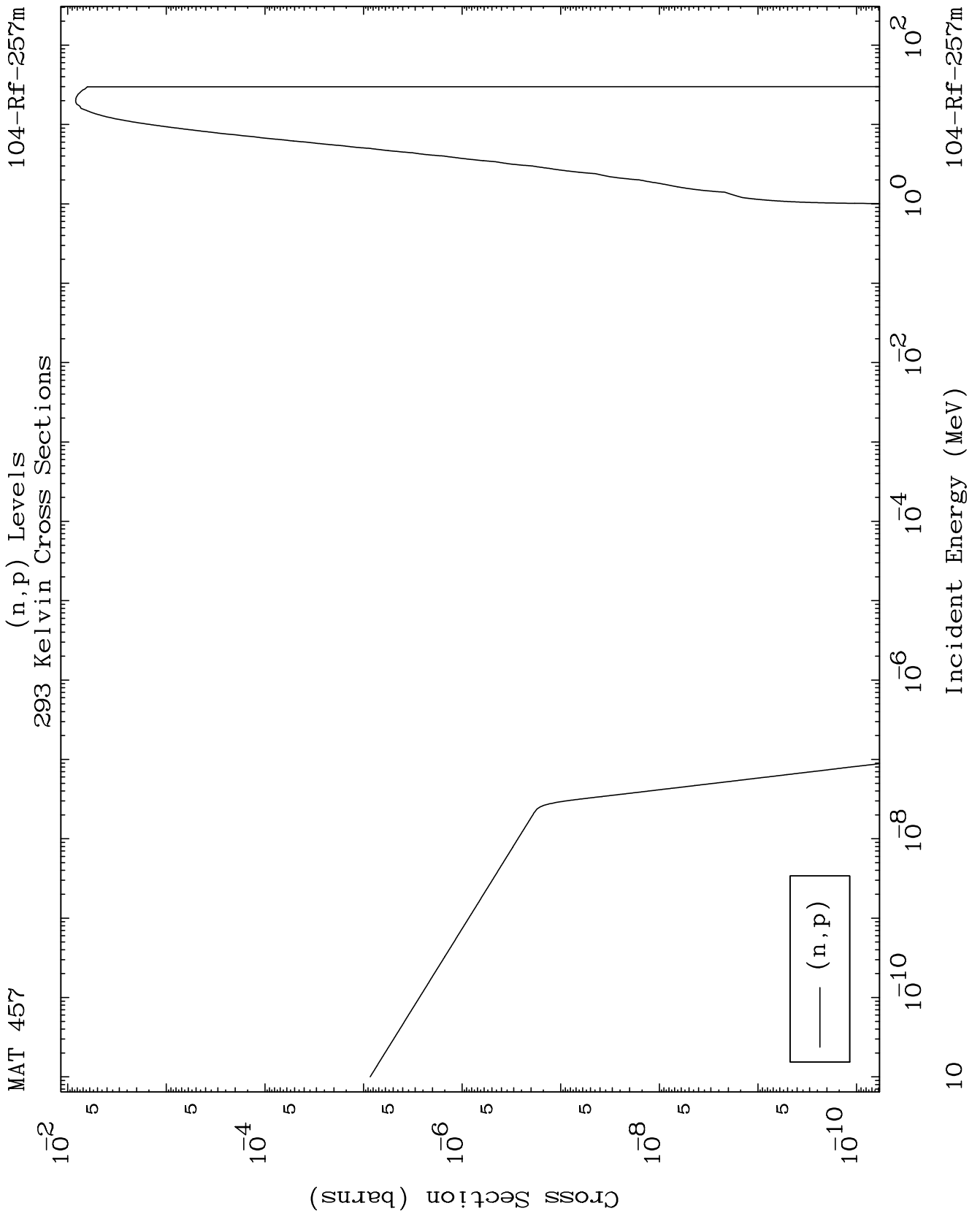


MAT 457

293 Kelvin Cross Sections
(n,n') Levels

104-Rf-257m

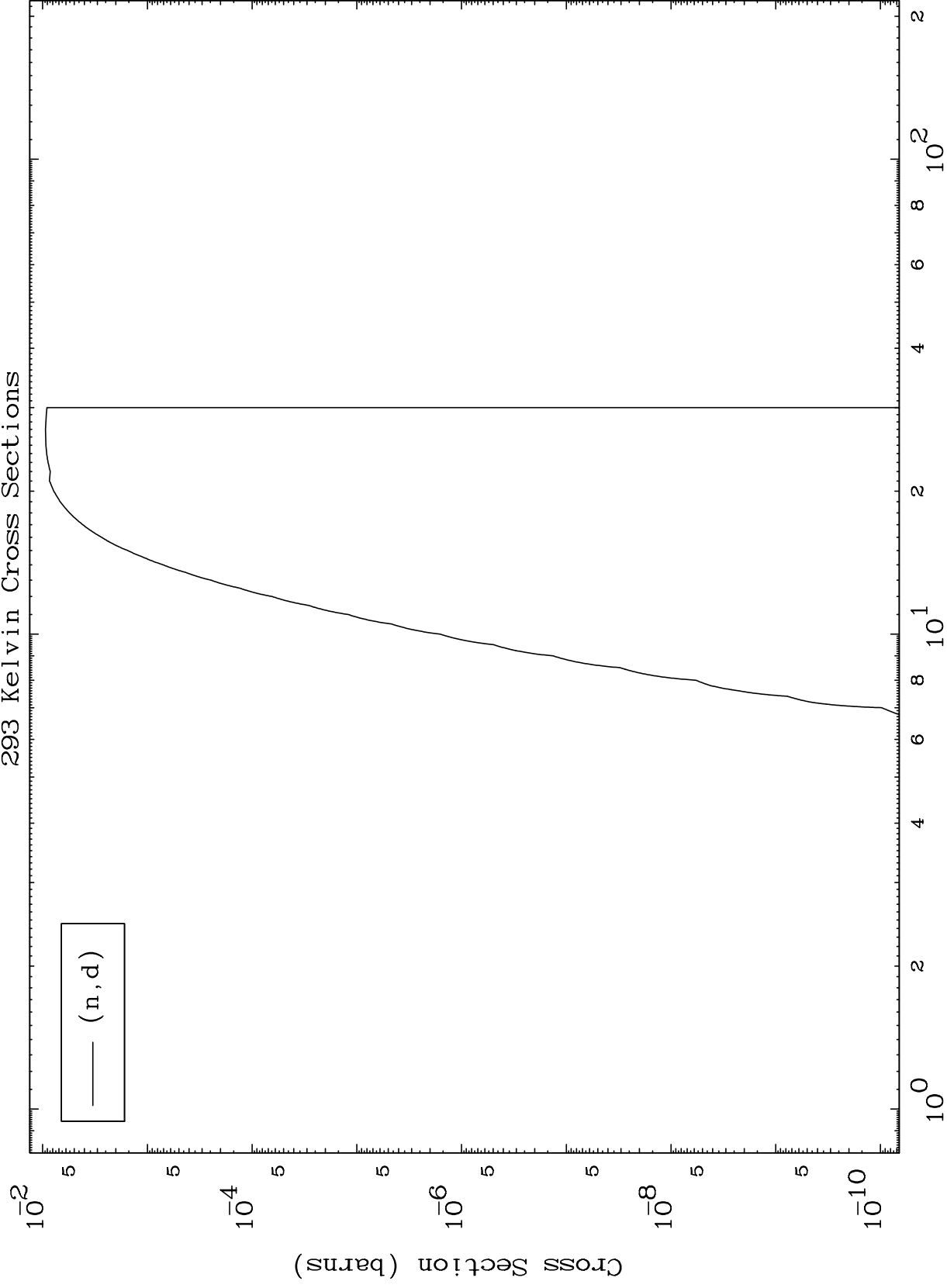




MAT 457

(n,d) Levels
293 Kelvin Cross Sections

104-Rf-257m



11

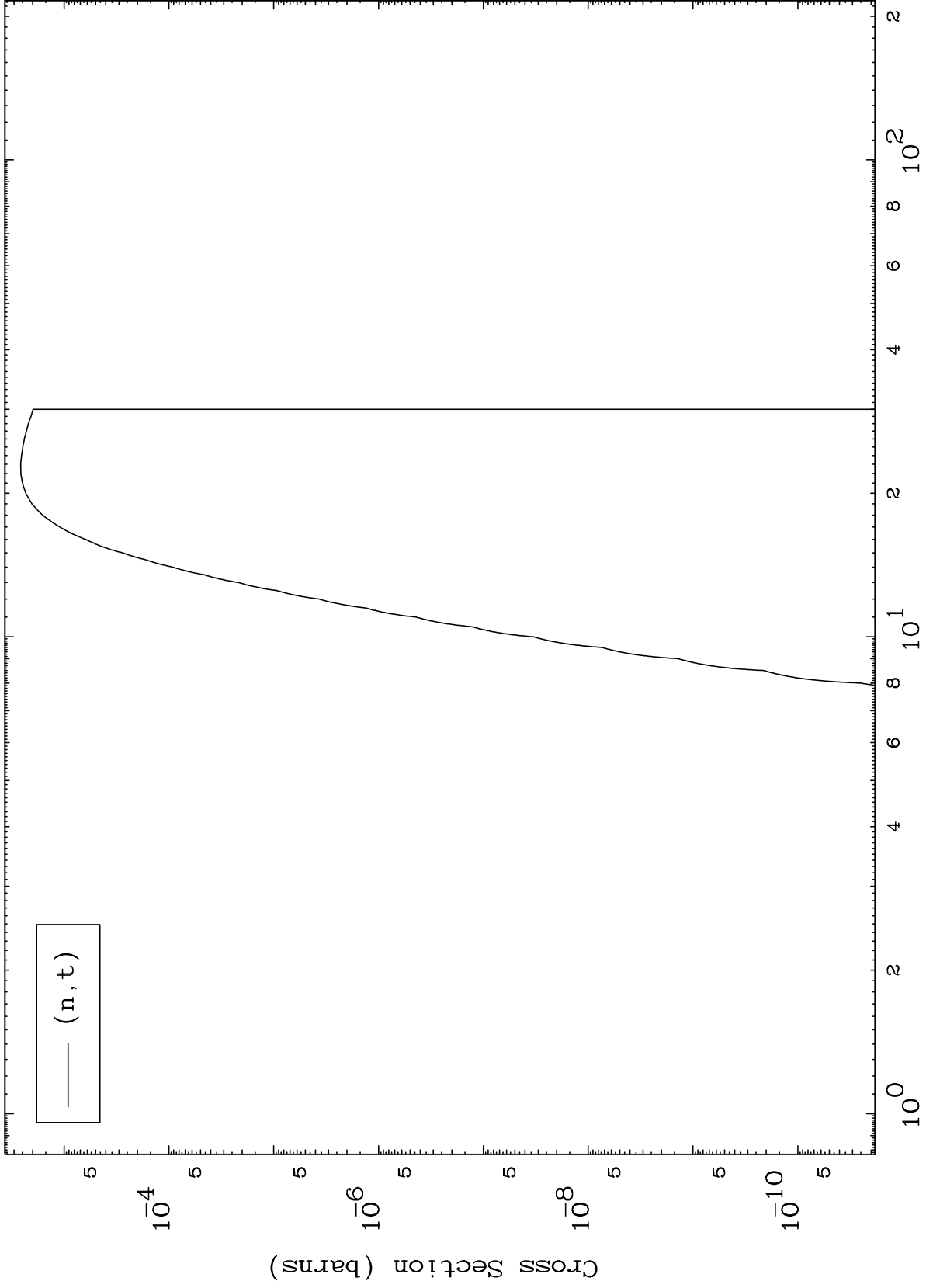
Incident Energy (MeV)

104-Rf-257m

MAT 457

(n, t) Levels
293 Kelvin Cross Sections

104-Rf-257m



12

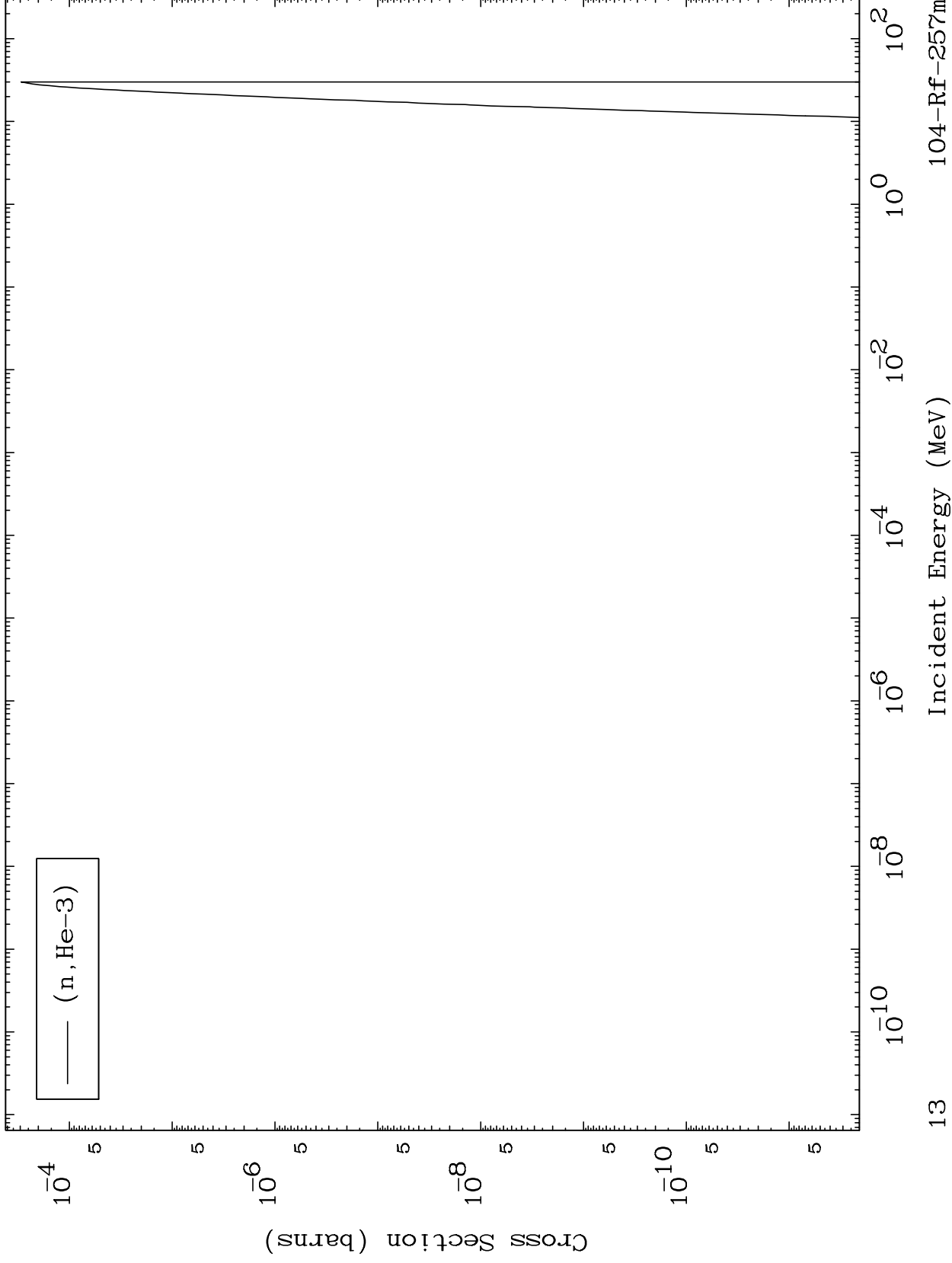
Incident Energy (MeV)

104-Rf-257m

MAT 457

(n,He3) Levels
293 Kelvin Cross Sections

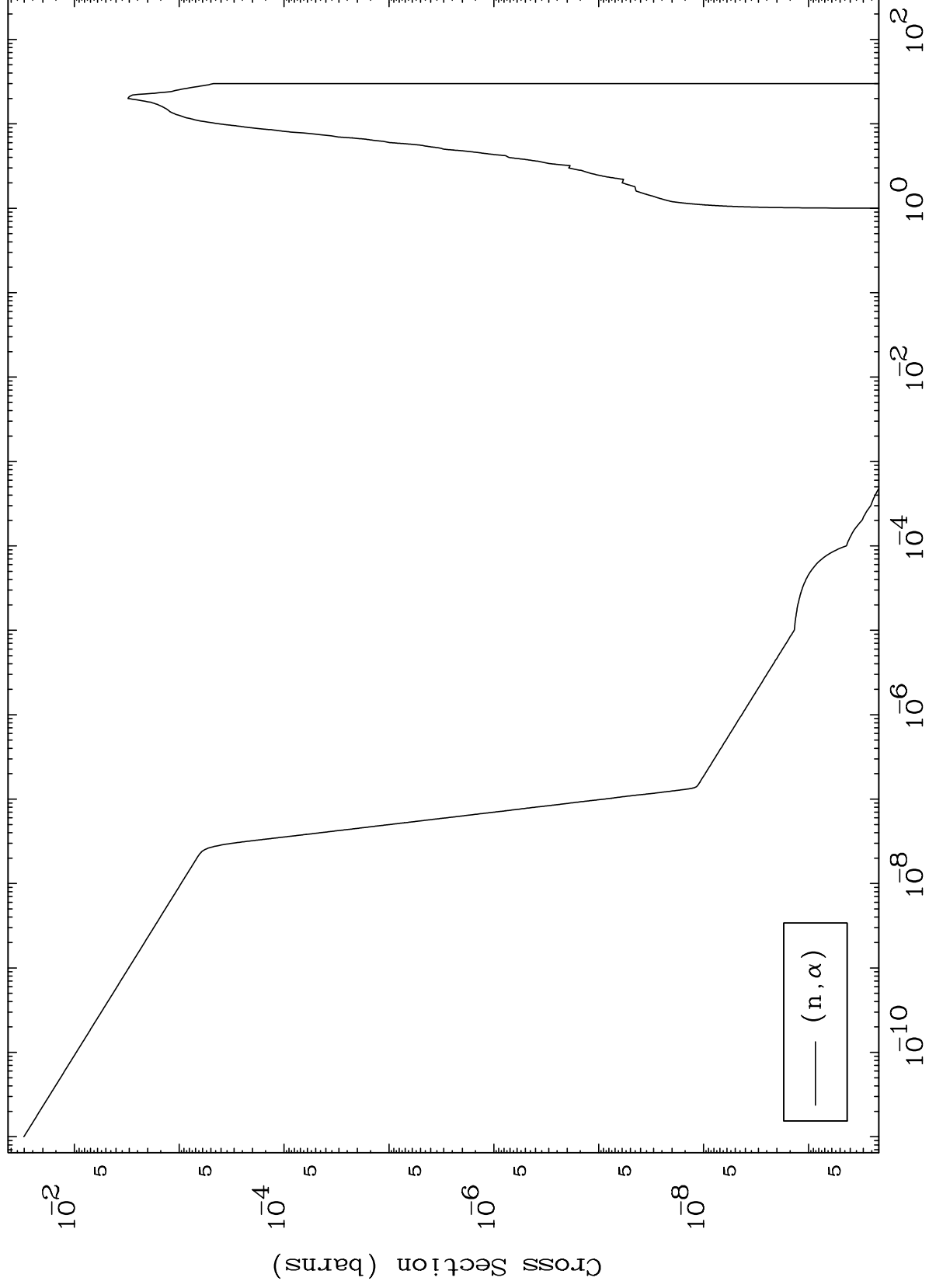
104-Rf-257m



MAT 457

(n, α) Levels
293 Kelvin Cross Sections

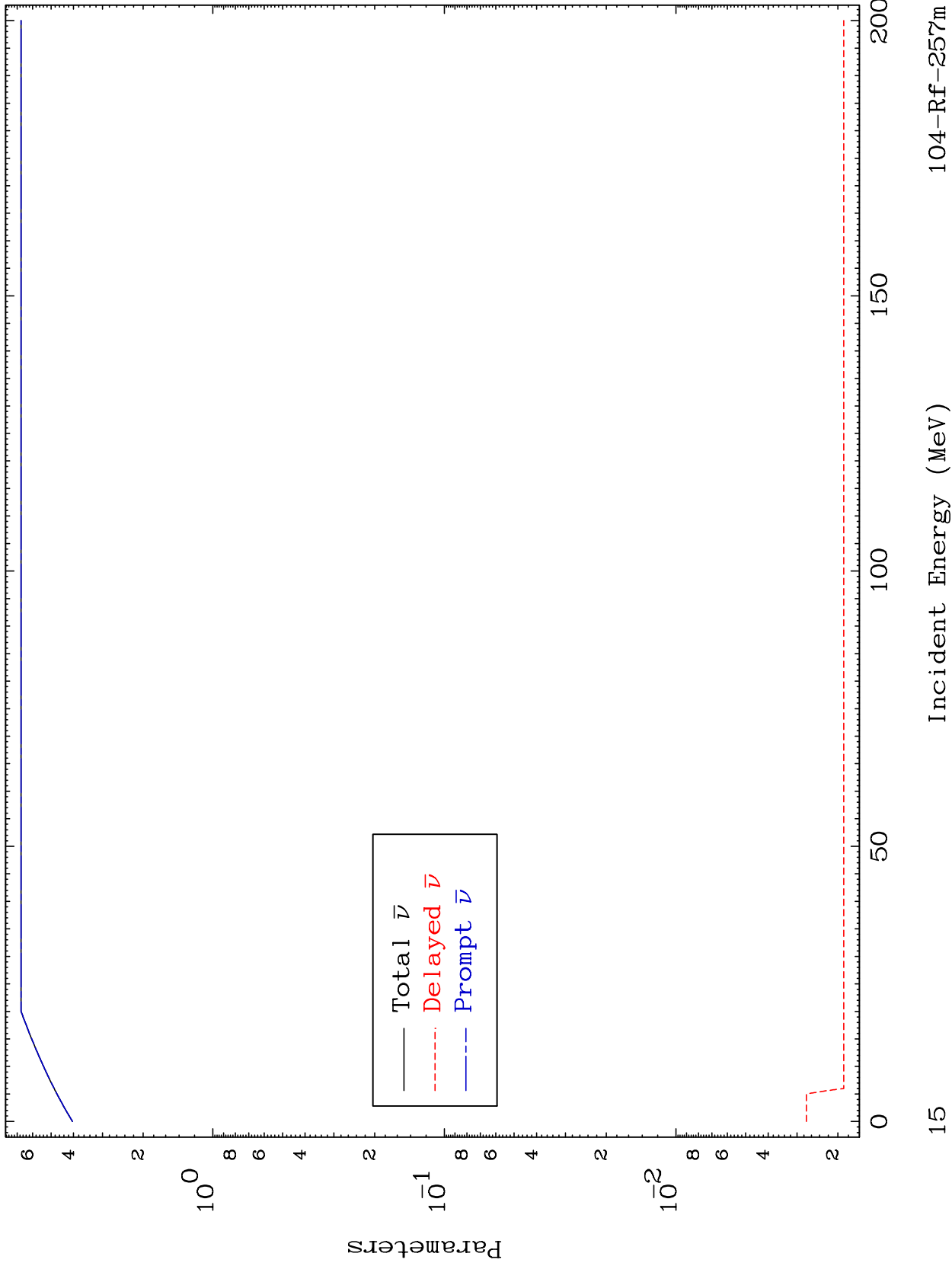
104-Rf-257m



MAT 457

Energy Release
Parameters

104-Rf-257m



0

50

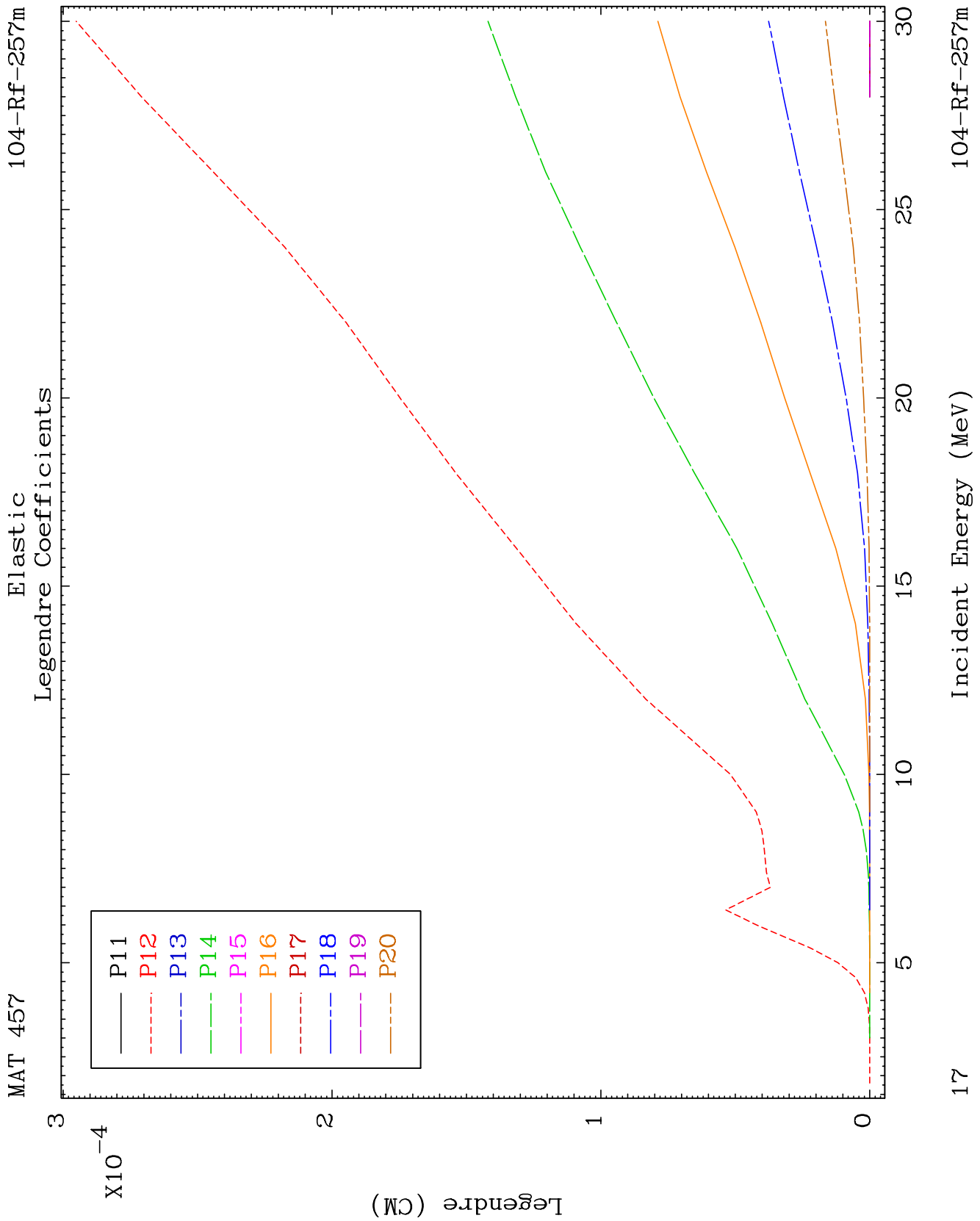
100

150

200

Incident Energy (MeV)

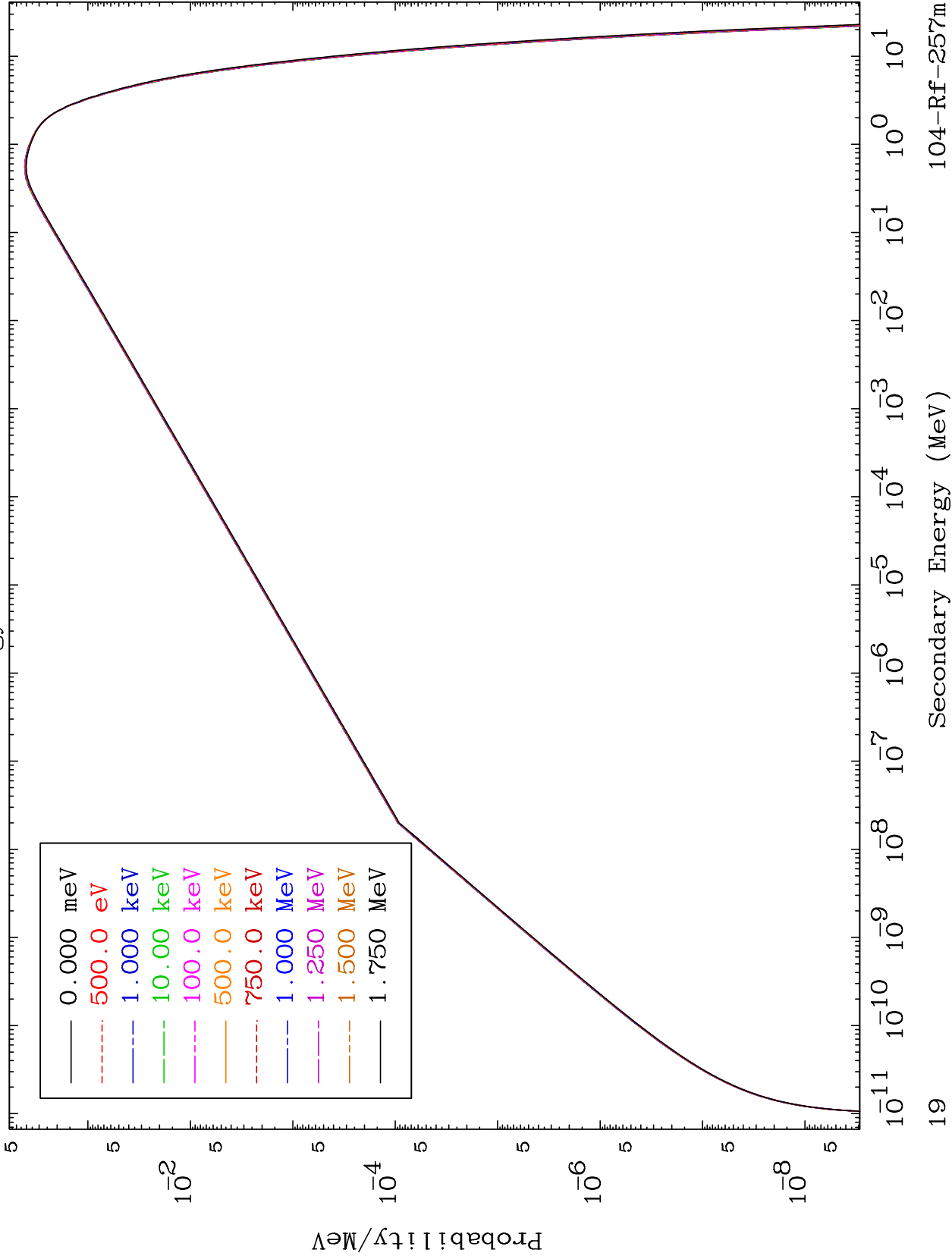
104-Rf-257m



MAT 457

Fission Energy Distributions

104-Rf-257m



104-Rf-257m

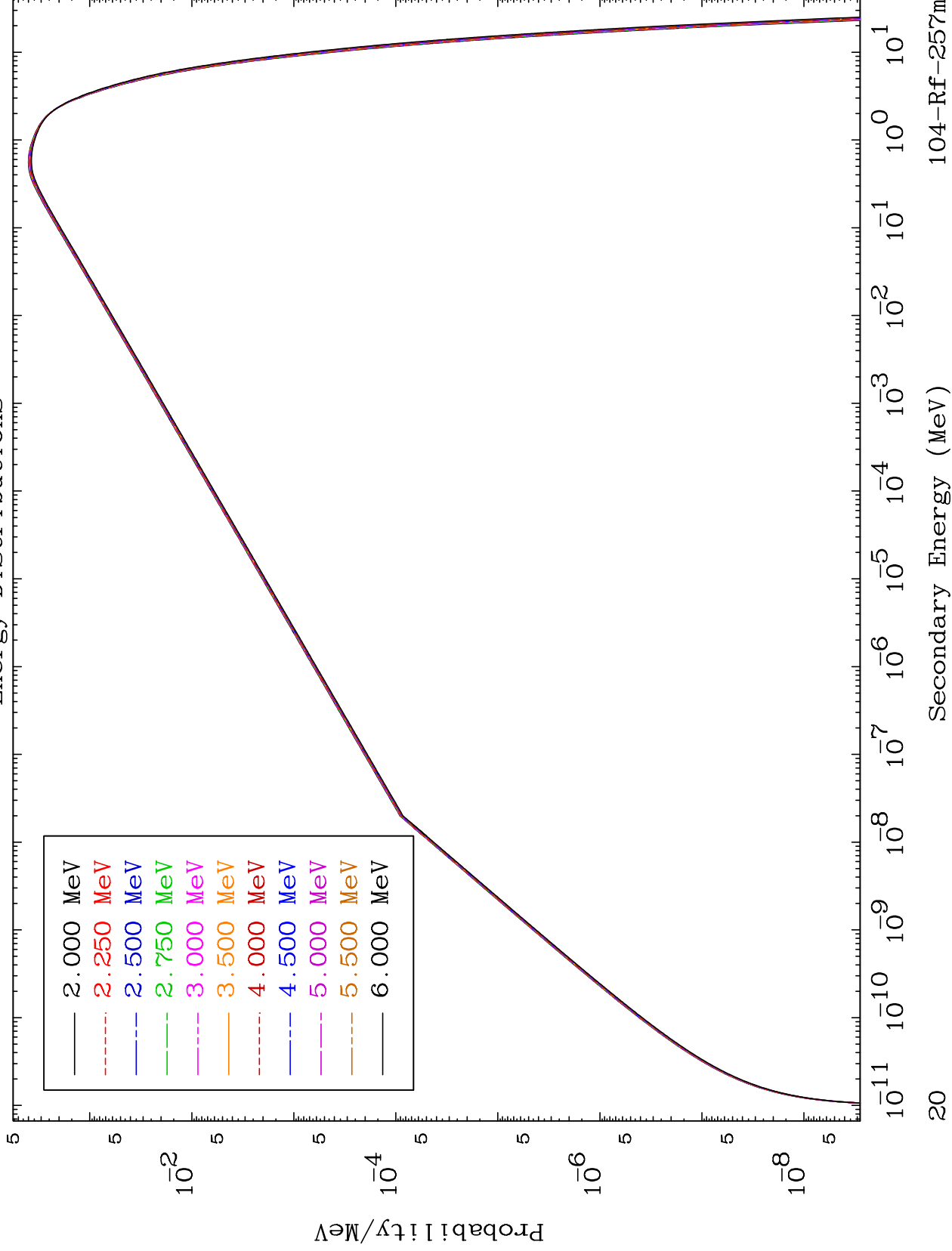
Secondary Energy (MeV)

19

MAT 457

Fission Energy Distributions

104-Rf-257m



104-Rf-257m

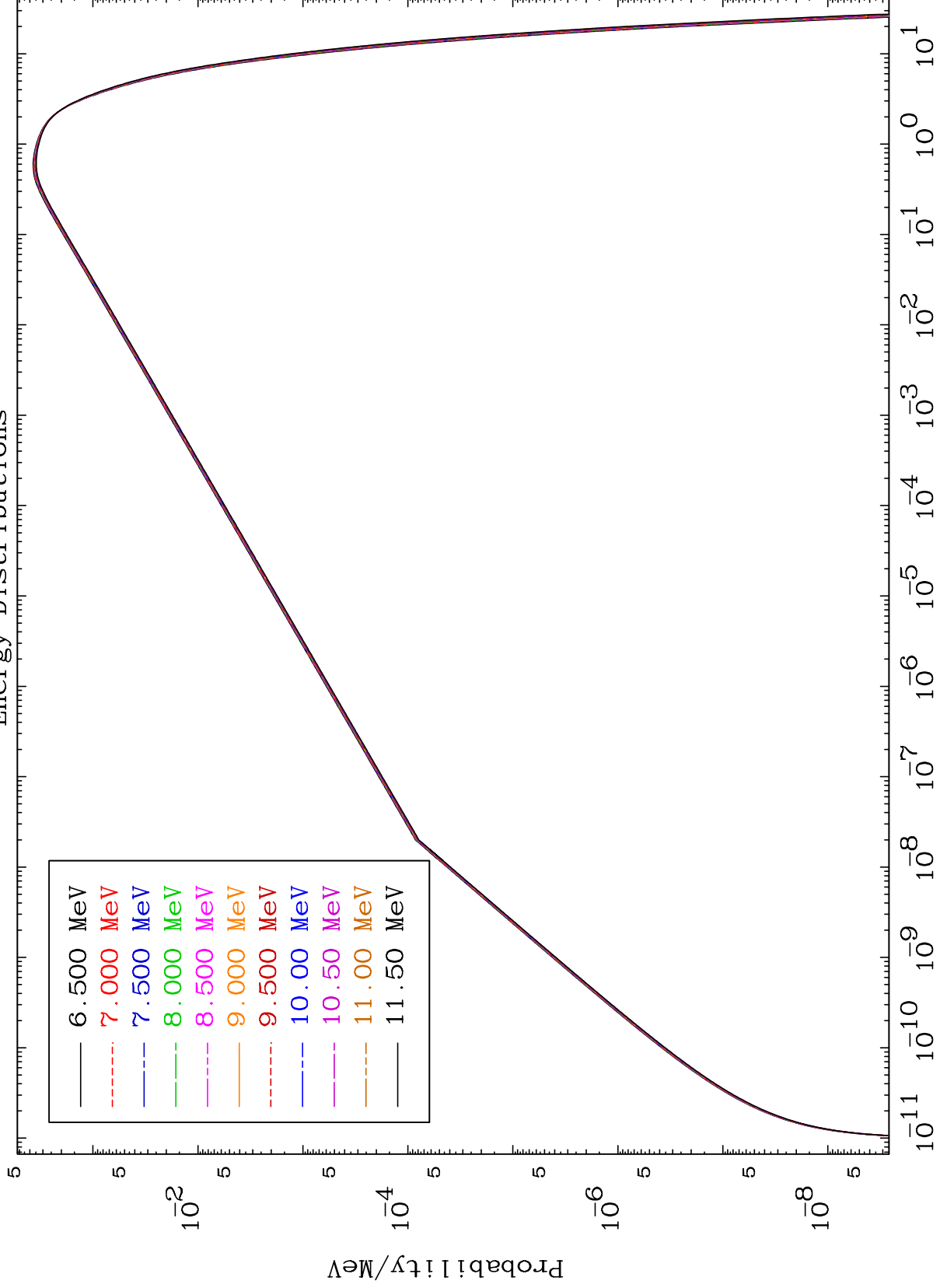
Secondary Energy (MeV)

20

MAT 457

Fission Energy Distributions

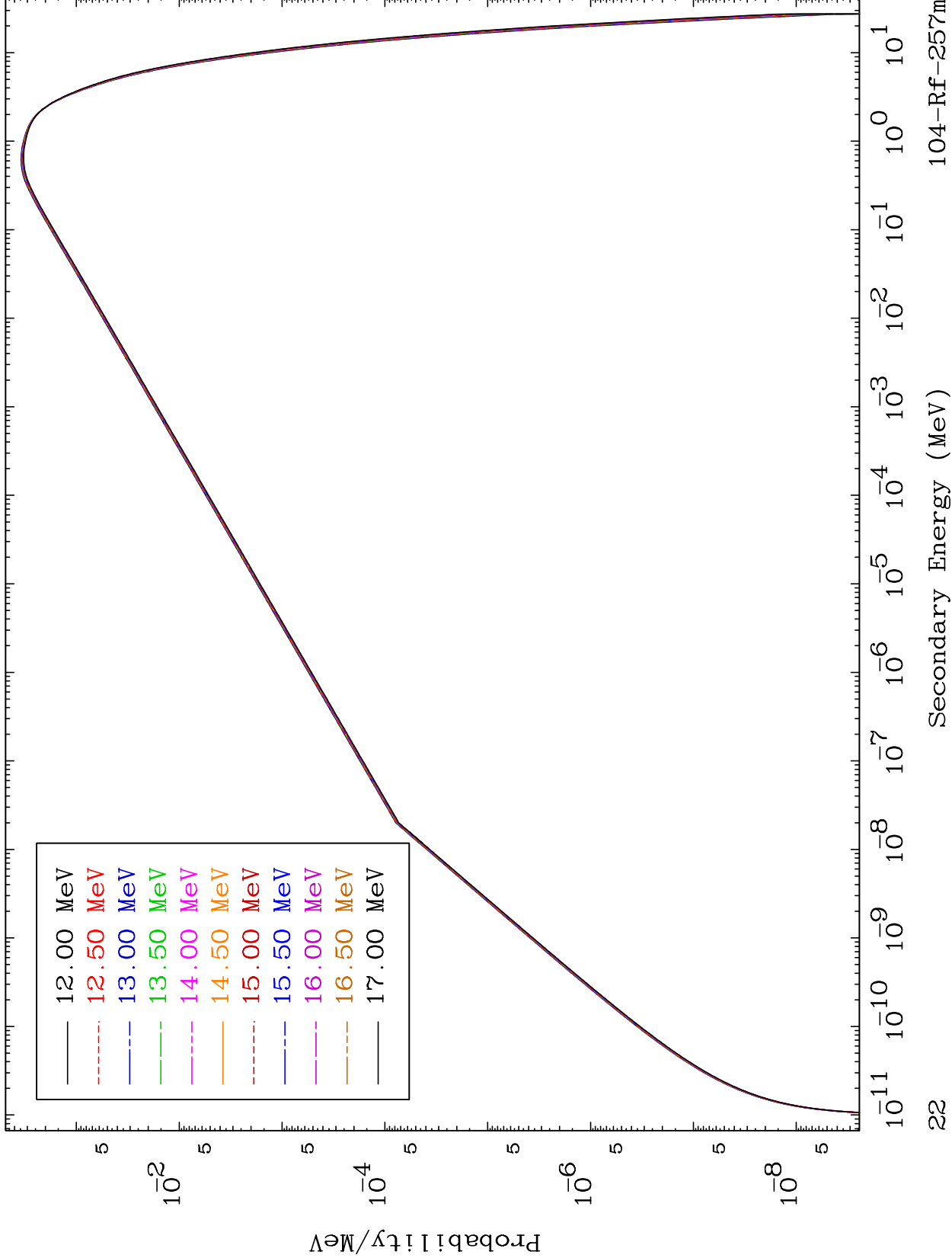
104-Rf-257m



MAT 457

Fission Energy Distributions

104-Rf-257m

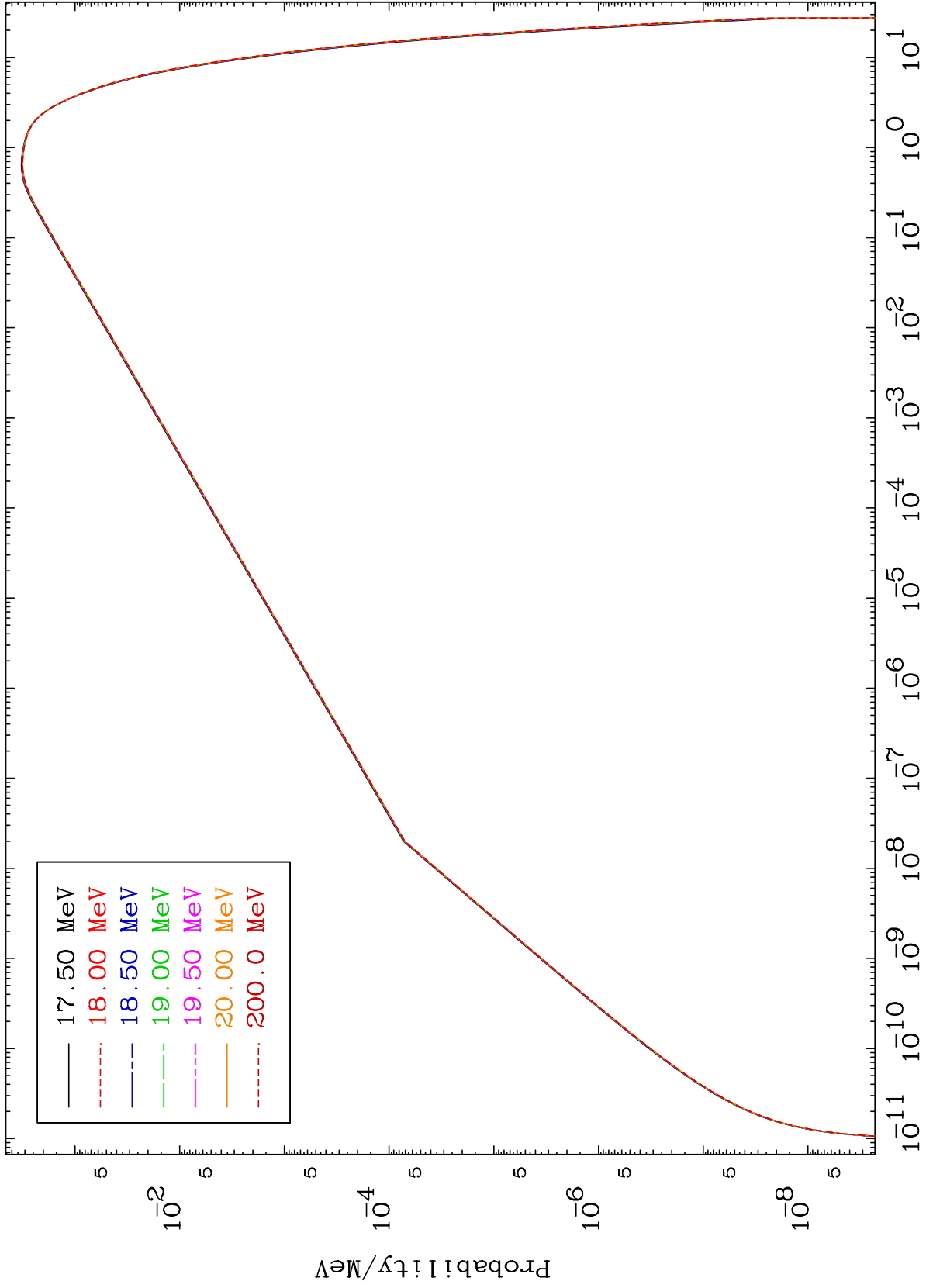


104-Rf-257m

MAT 457

Fission Energy Distributions

104-Rf-257m



23

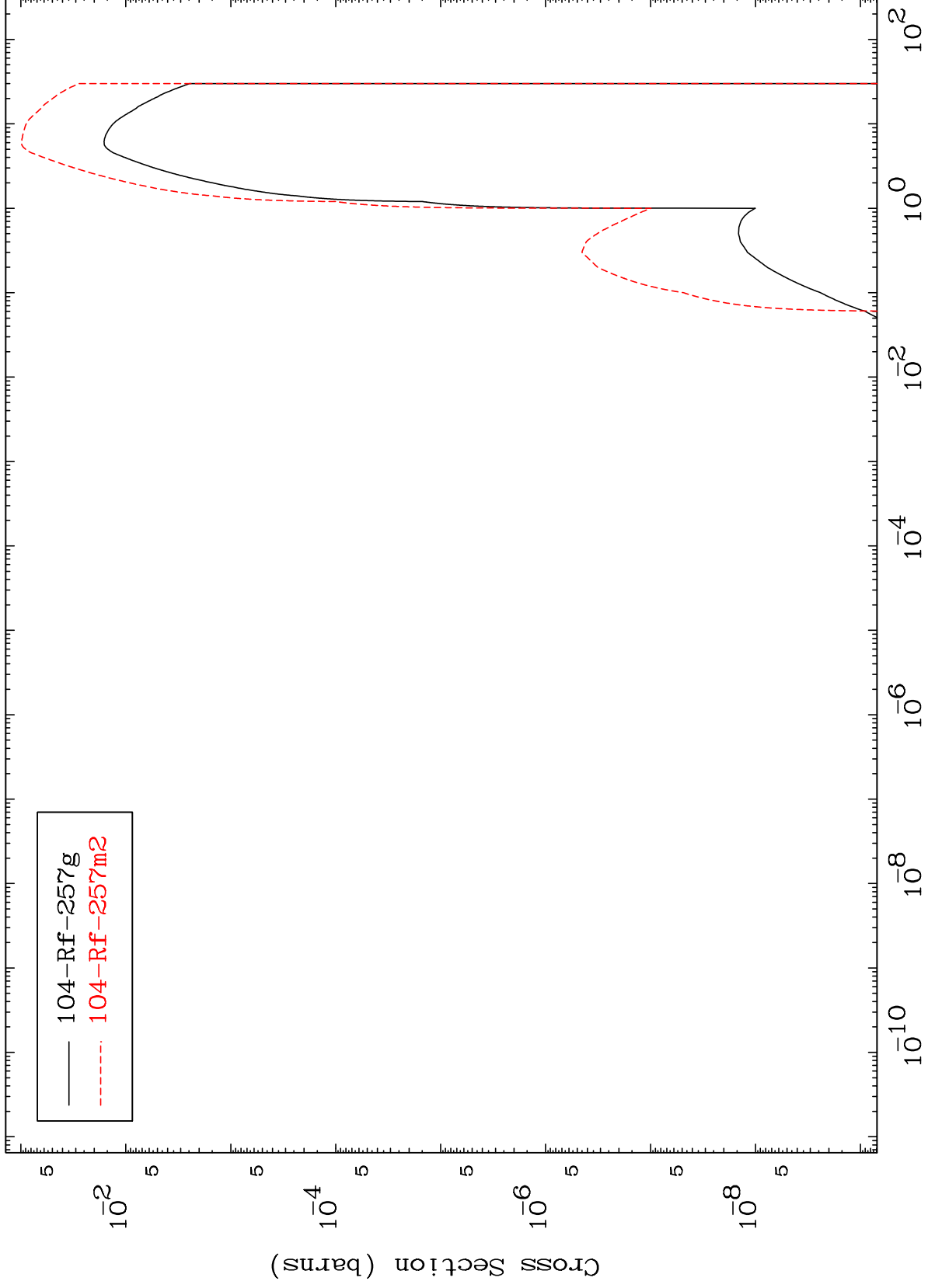
Secondary Energy (MeV)

104-Rf-257m

MAT 457

Inelastic
Radionuclide Production Cross Section

104-Rf-257m



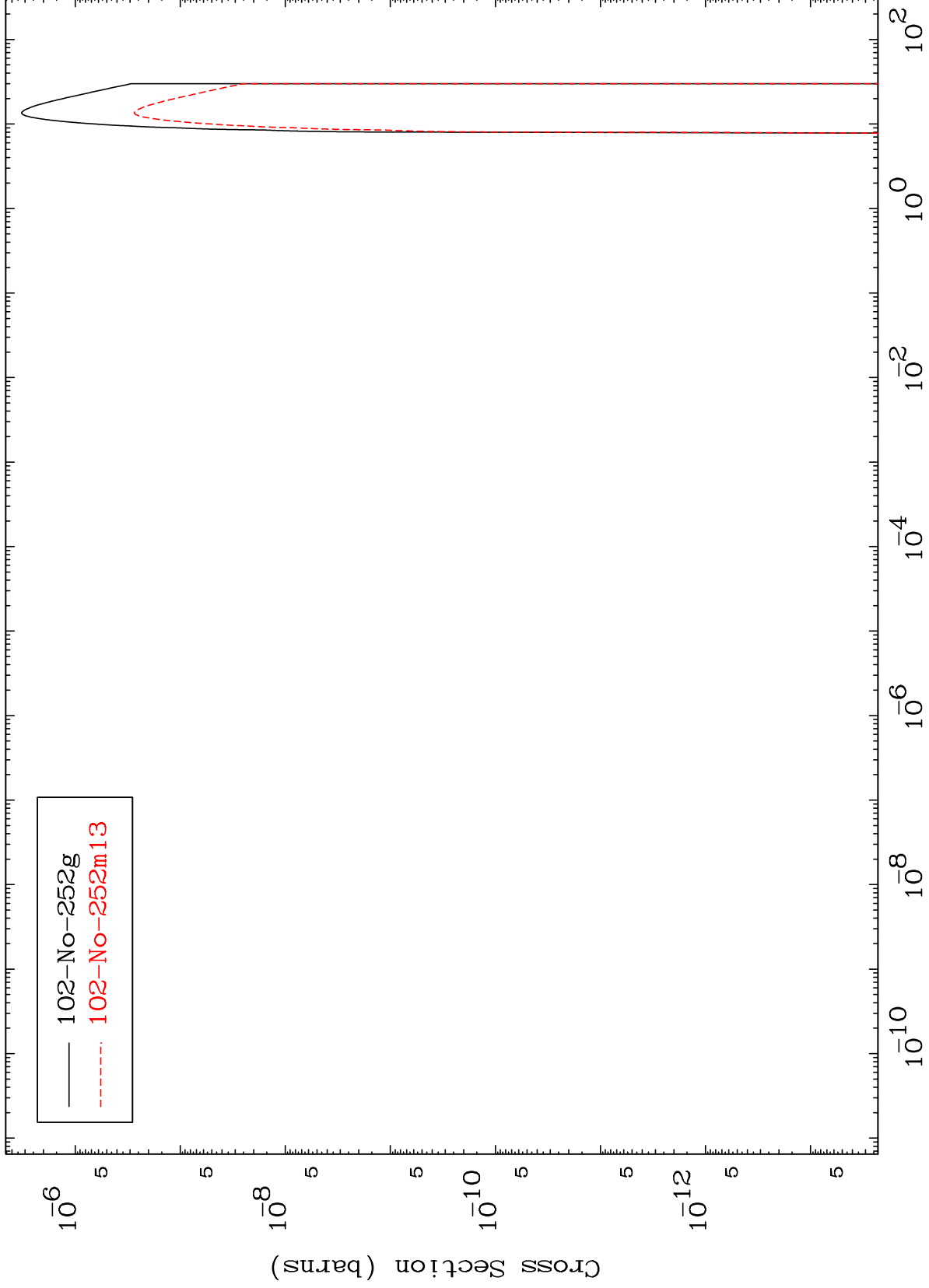
— 104-Rf-257g
- - - 104-Rf-257m2

MAT 457

(n,2n) α

104-Rf-257m

Radionuclide Production Cross Section



25

Incident Energy (MeV)

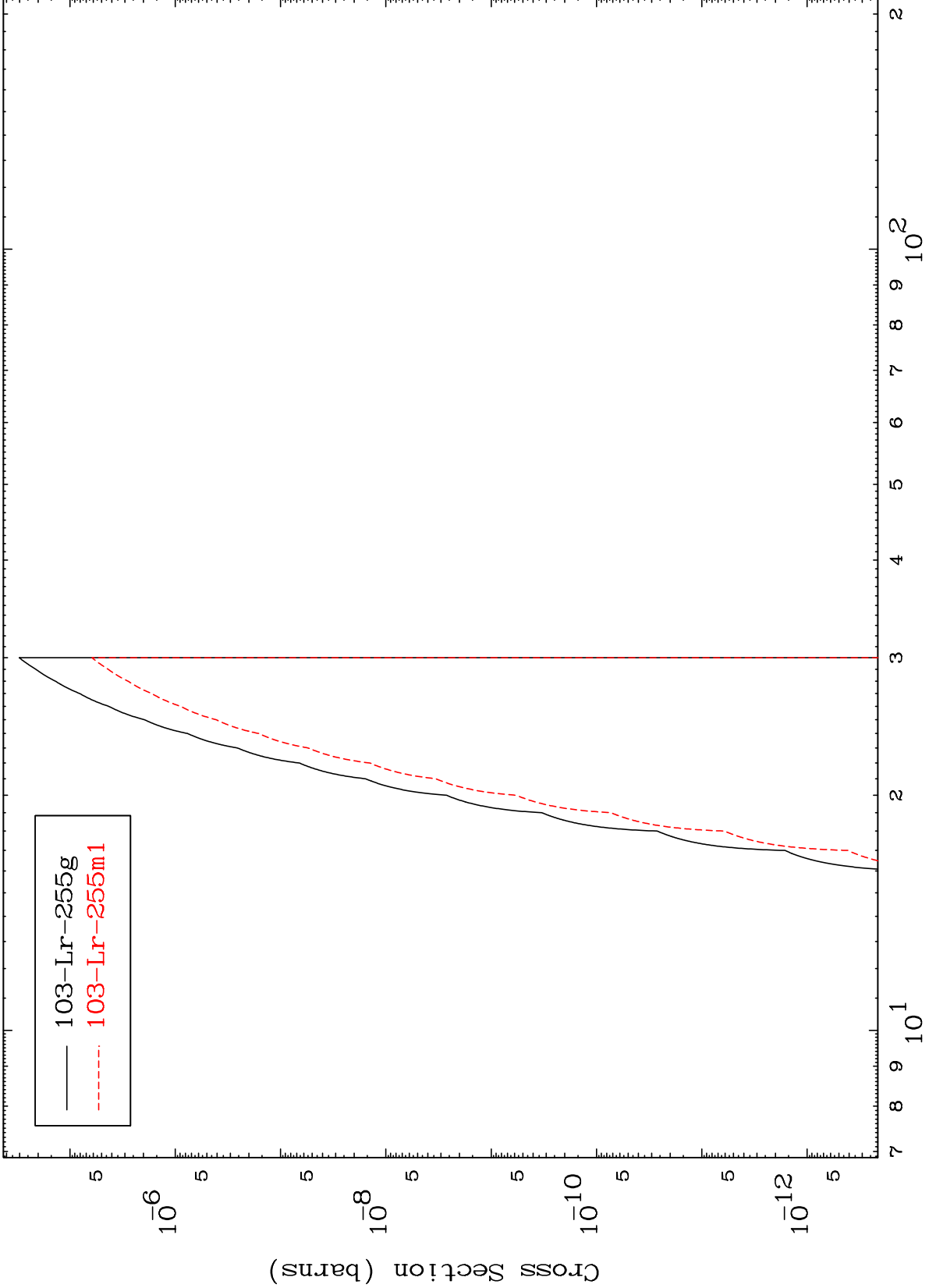
104-Rf-257m

MAT 457

(n,n') d

104-Rf-257m

Radionuclide Production Cross Section



26

Incident Energy (MeV)

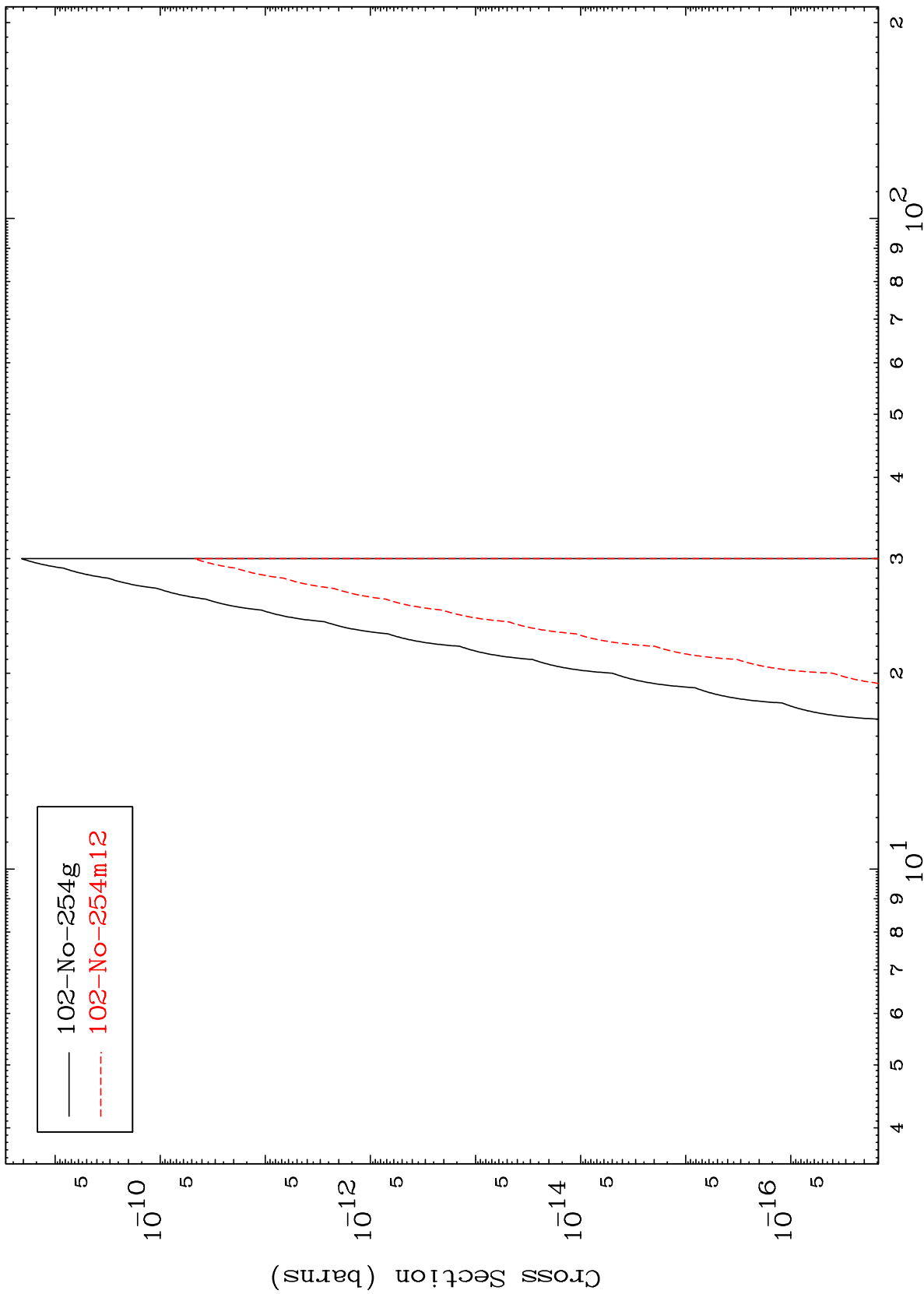
104-Rf-257m

MAT 457

(n,n') He-3

104-Rf-257m

Radionuclide Production Cross Section



Incident Energy (MeV)

104-Rf-257m

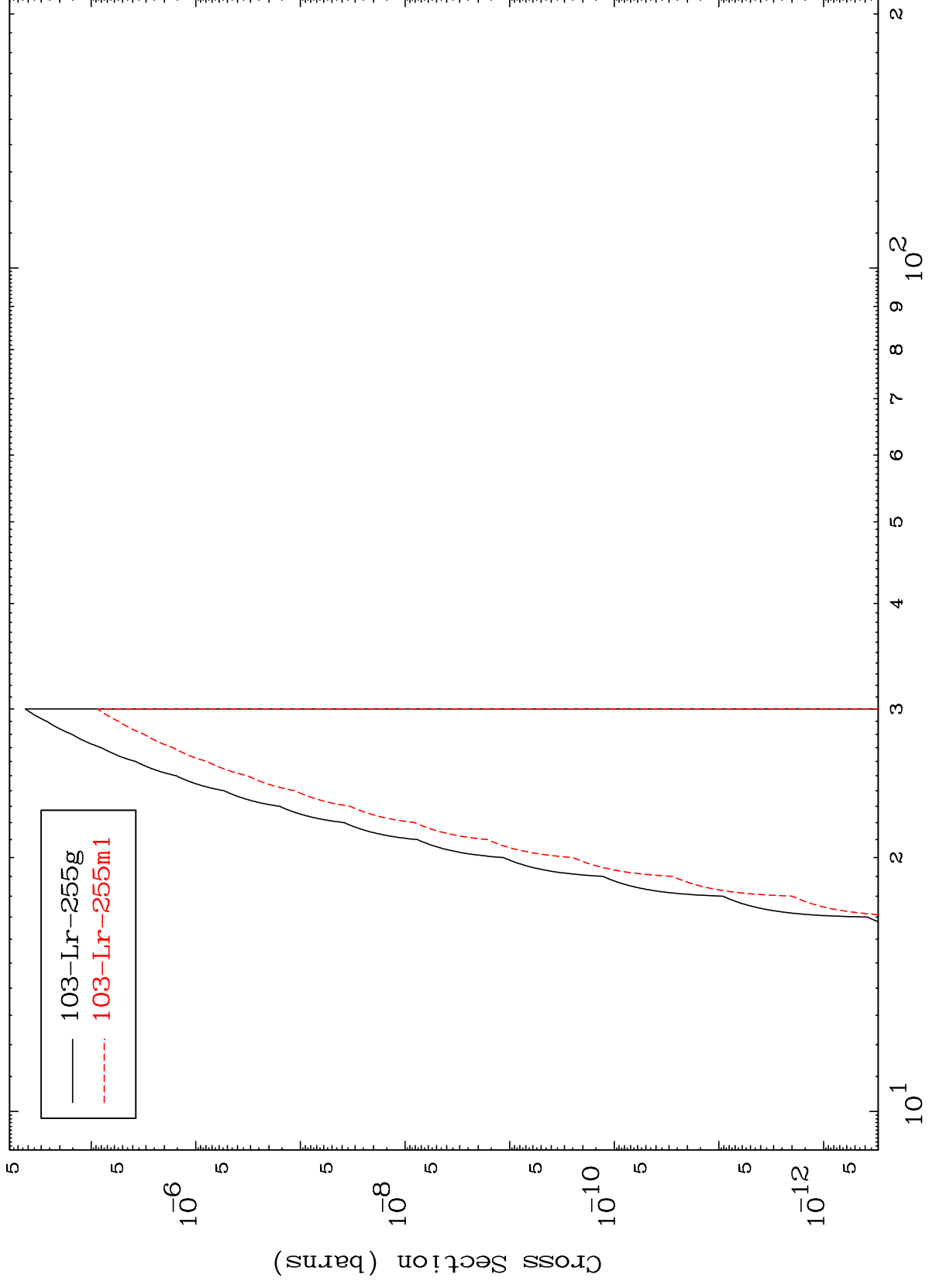
27

MAT 457

(n,2n) p

104-Rf-257m

Radionuclide Production Cross Section



— 103-Lr-255g
- - - 103-Lr-255m1

Incident Energy (MeV)

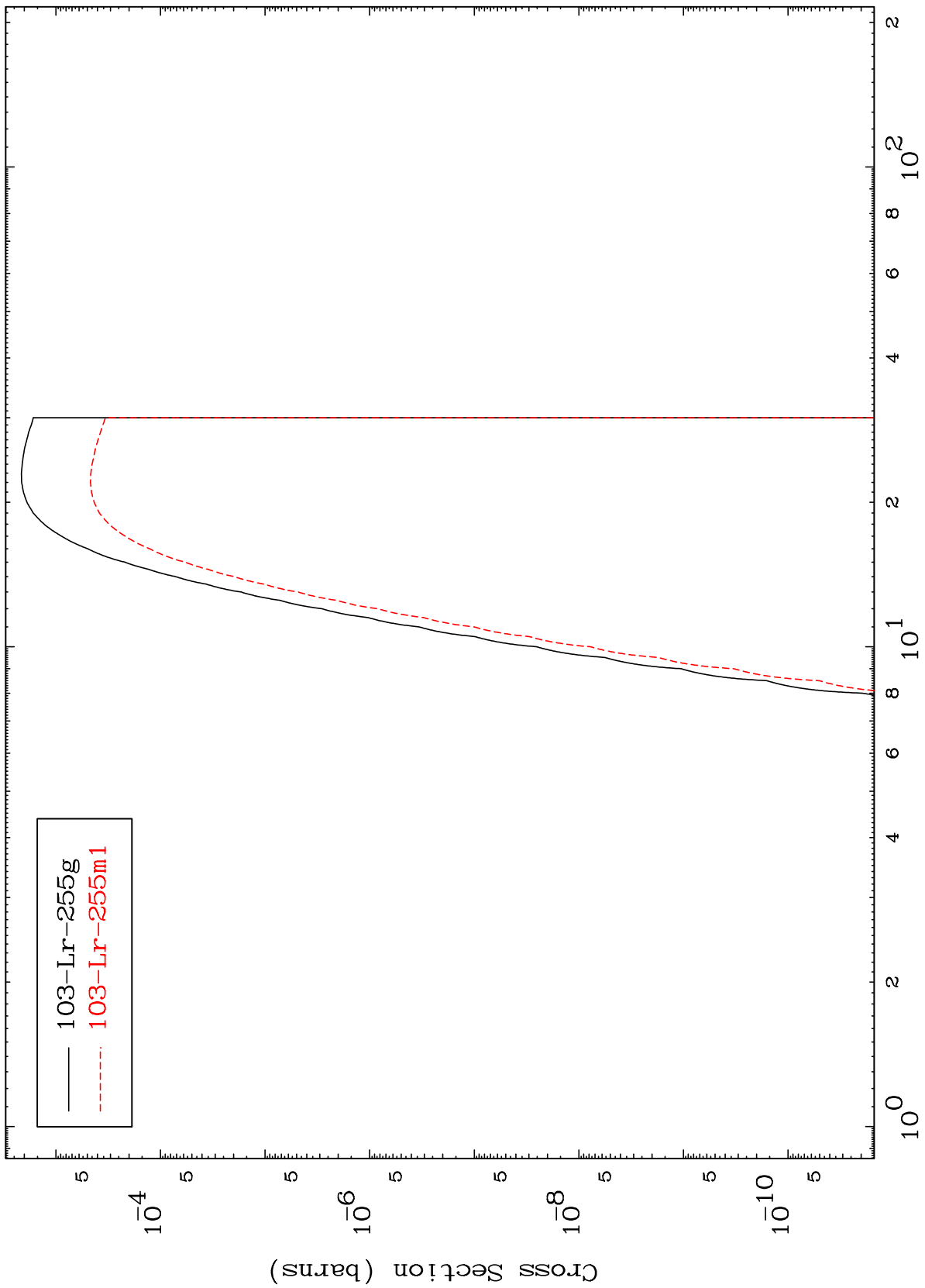
104-Rf-257m

28

MAT 457

104-Rf-257m

(n, t)
Radionuclide Production Cross Section



— 103-Lr-255g
- - - 103-Lr-255m1

104-Rf-257m

Incident Energy (MeV)

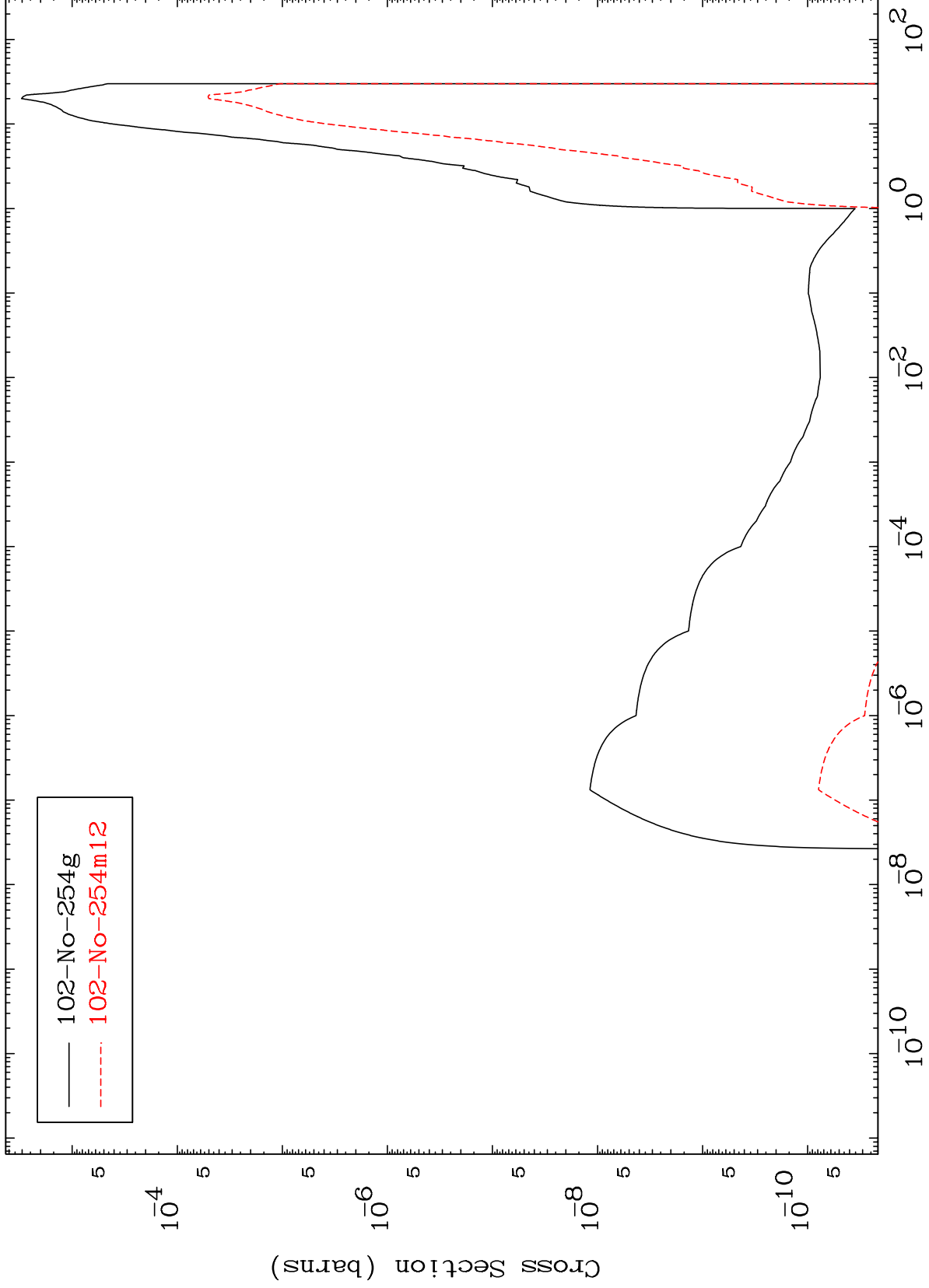
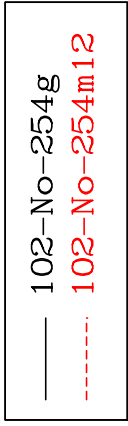
29

MAT 457

(n, α)

104-Rf-257m

Radionuclide Production Cross Section



30

Incident Energy (MeV)

104-Rf-257m