

Program EVALPLOT  
(Version 2021-1)

by

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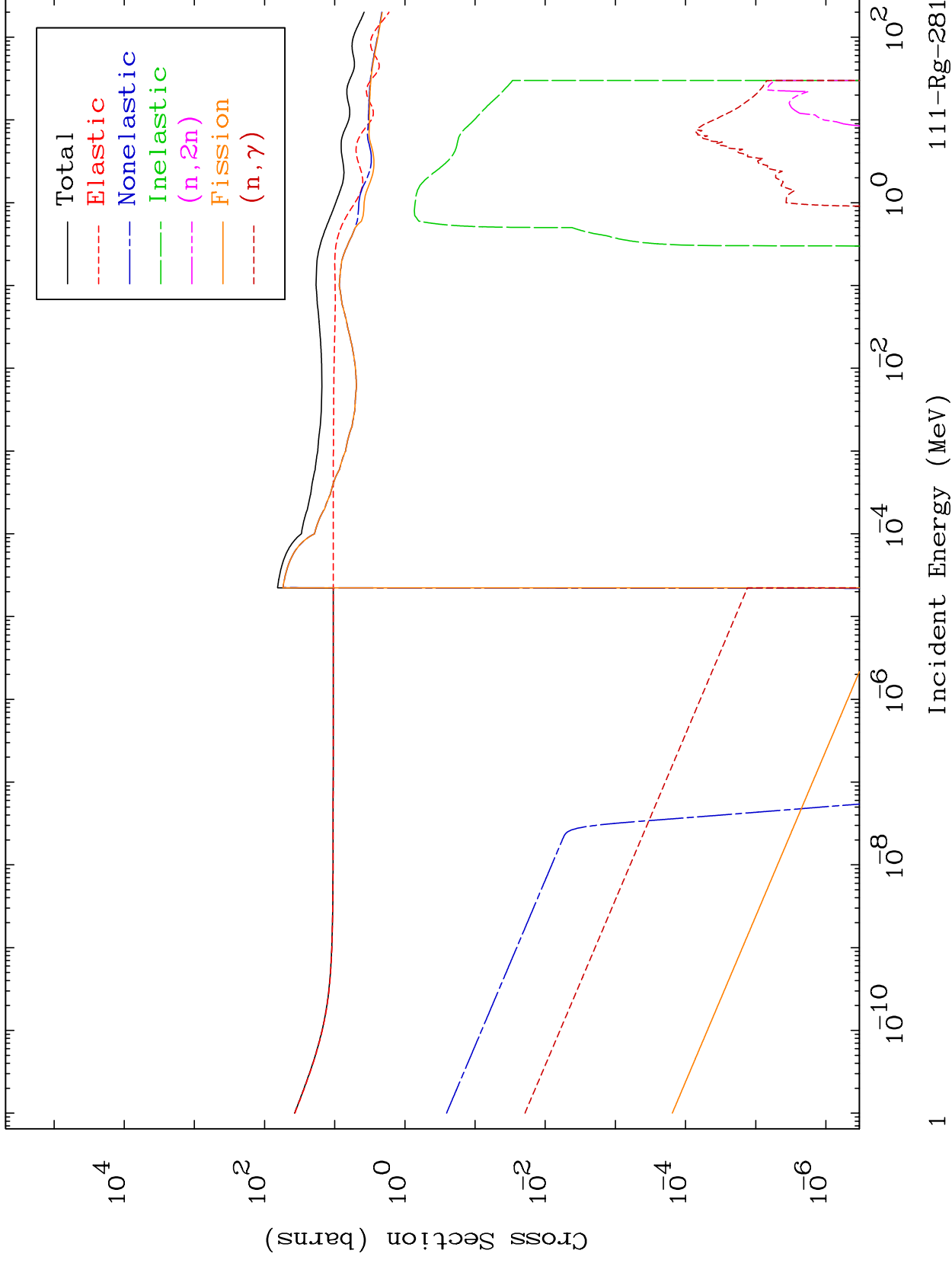
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 1181

Neutron Major  
293 Kelvin Cross Sections

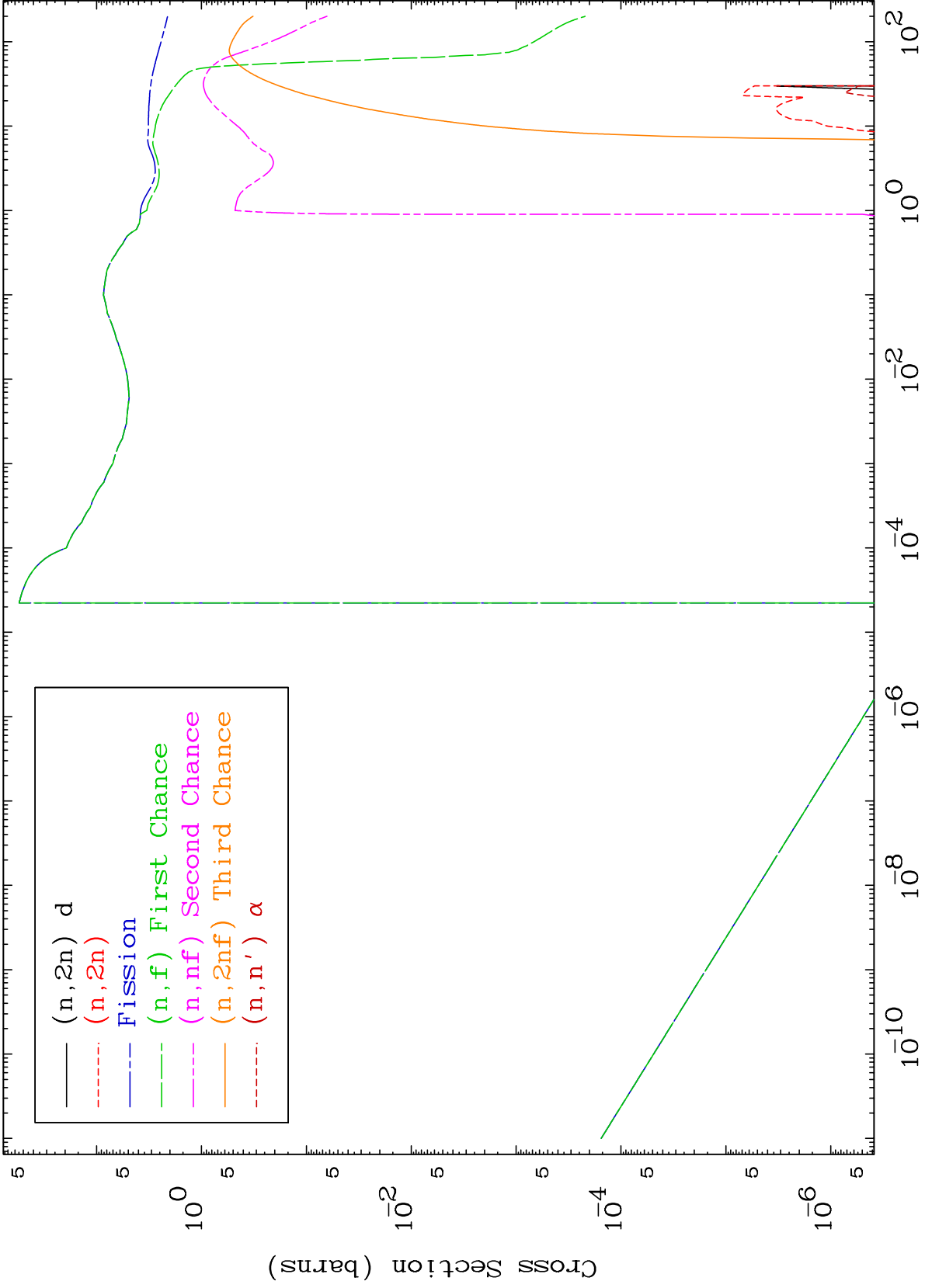
111-Rg-281

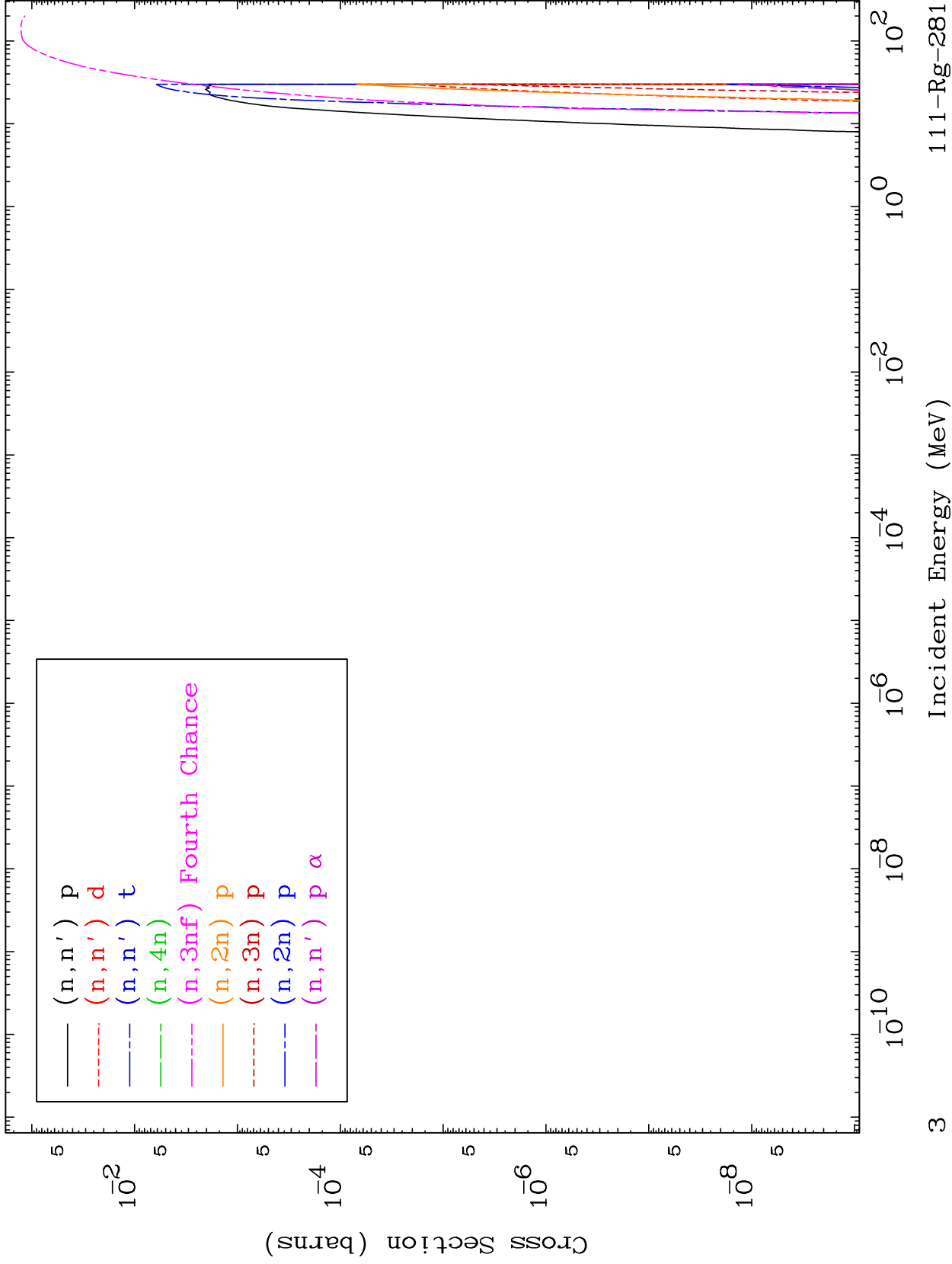


MAT 1181

Neutron Absorption  
293 Kelvin Cross Sections

111-Rg-281

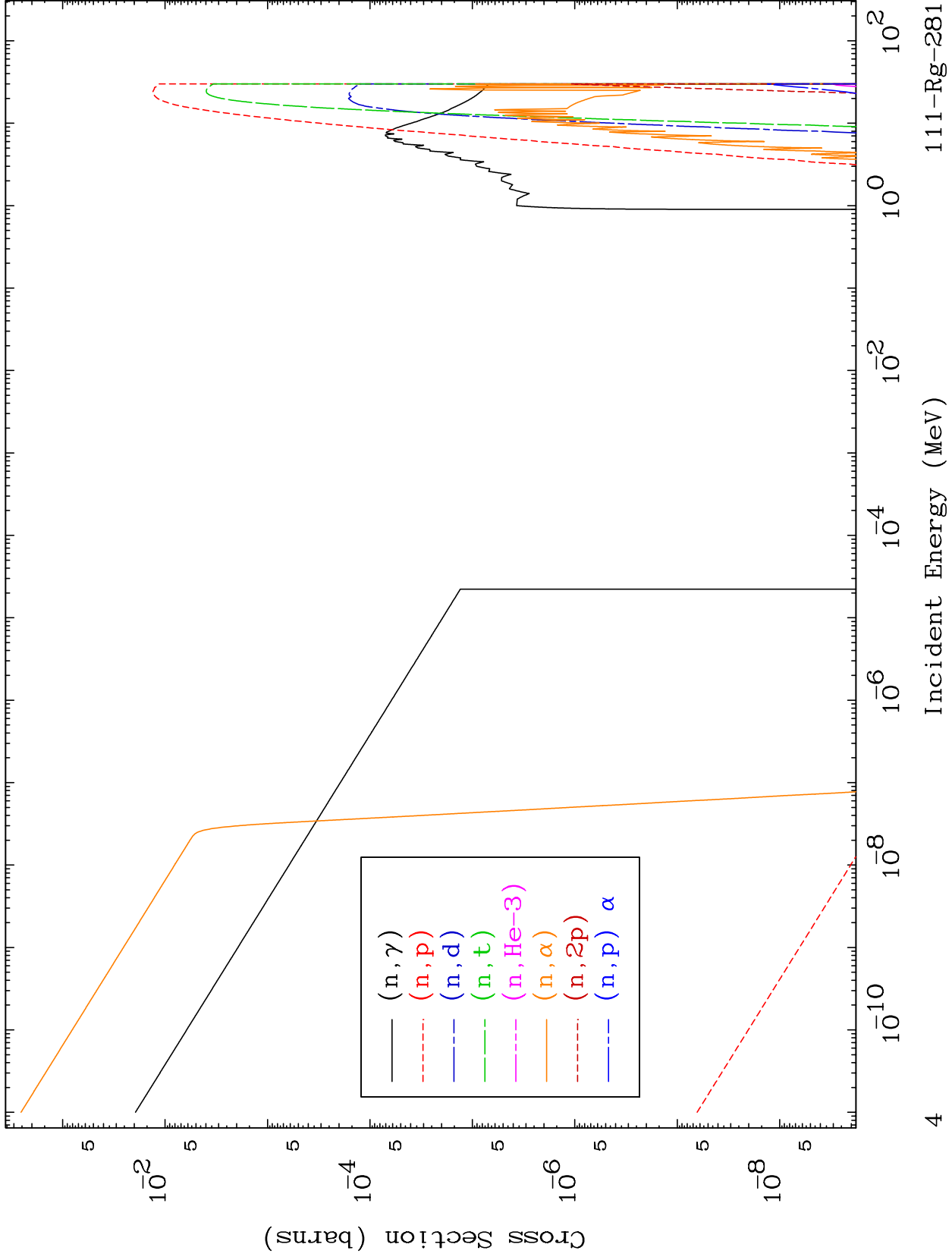




MAT 1181

Neutron Absorption  
293 Kelvin Cross Sections

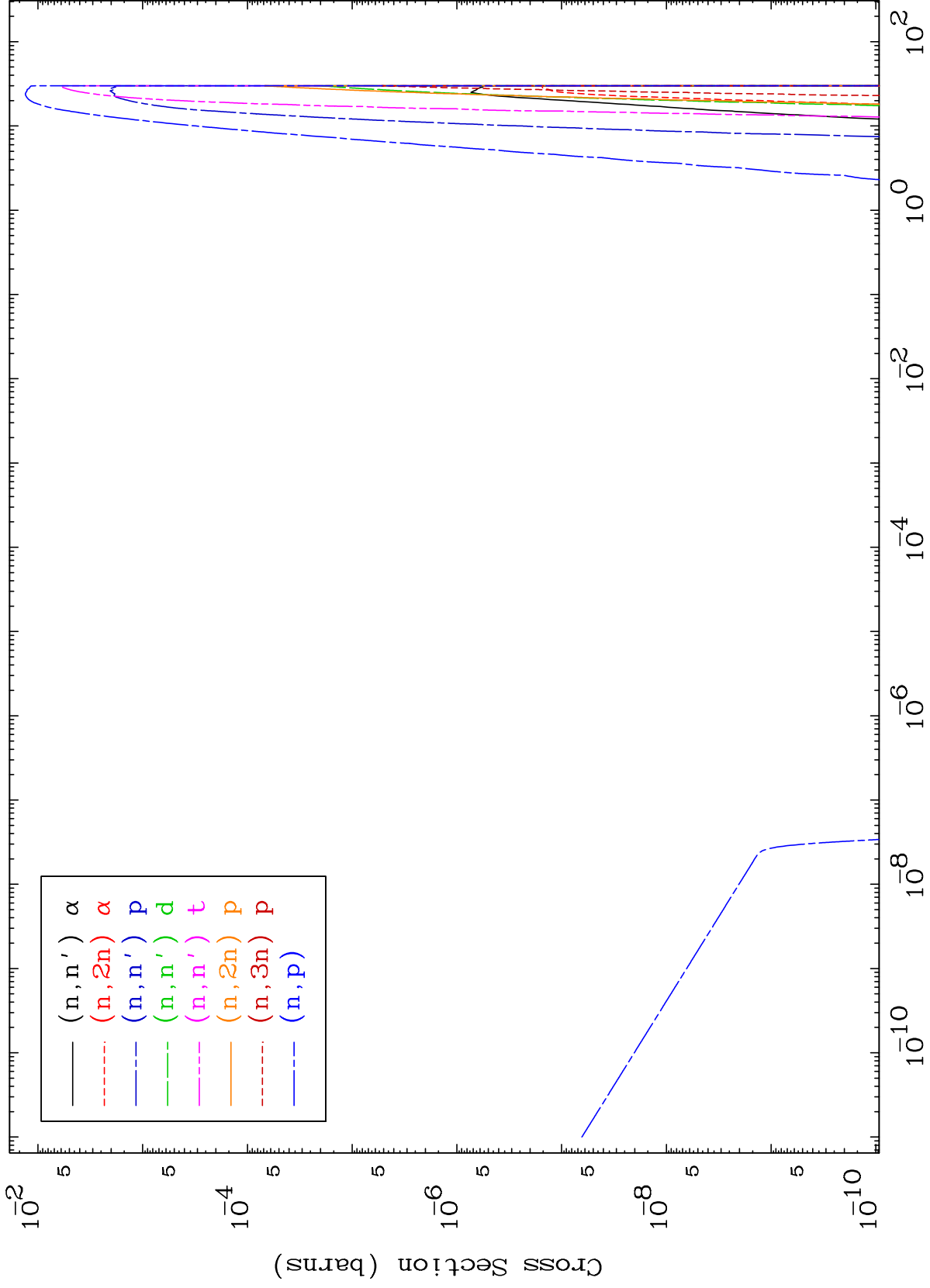
111-Rg-281



MAT 1181

Charged Particle  
293 Kelvin Cross Sections

111-Rg-281



5

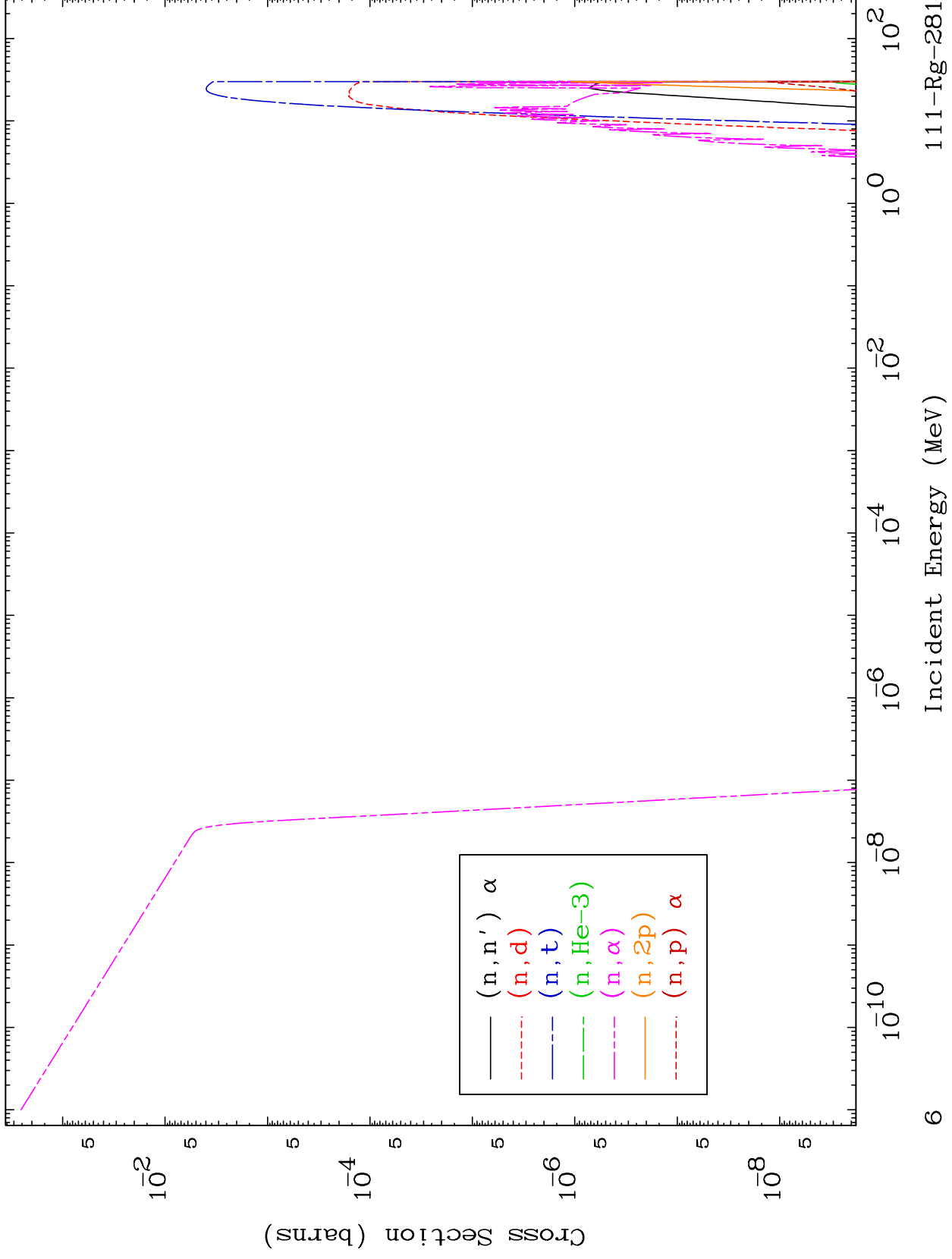
Incident Energy (MeV)

111-Rg-281

MAT 1181

Charged Particle  
293 Kelvin Cross Sections

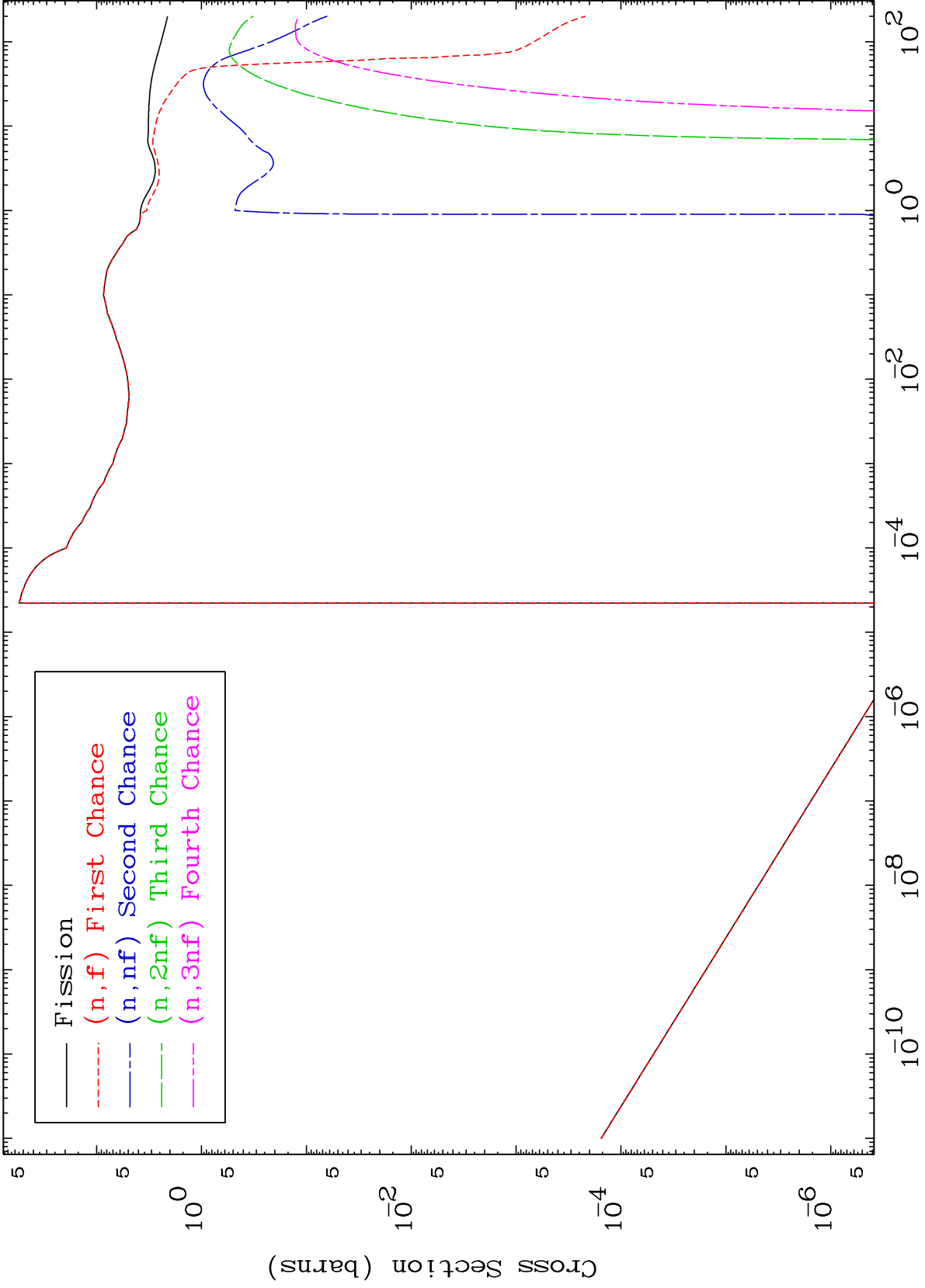
111-Rg-281



MAT 1181

Fission  
293 Kelvin Cross Sections

111-Rg-281

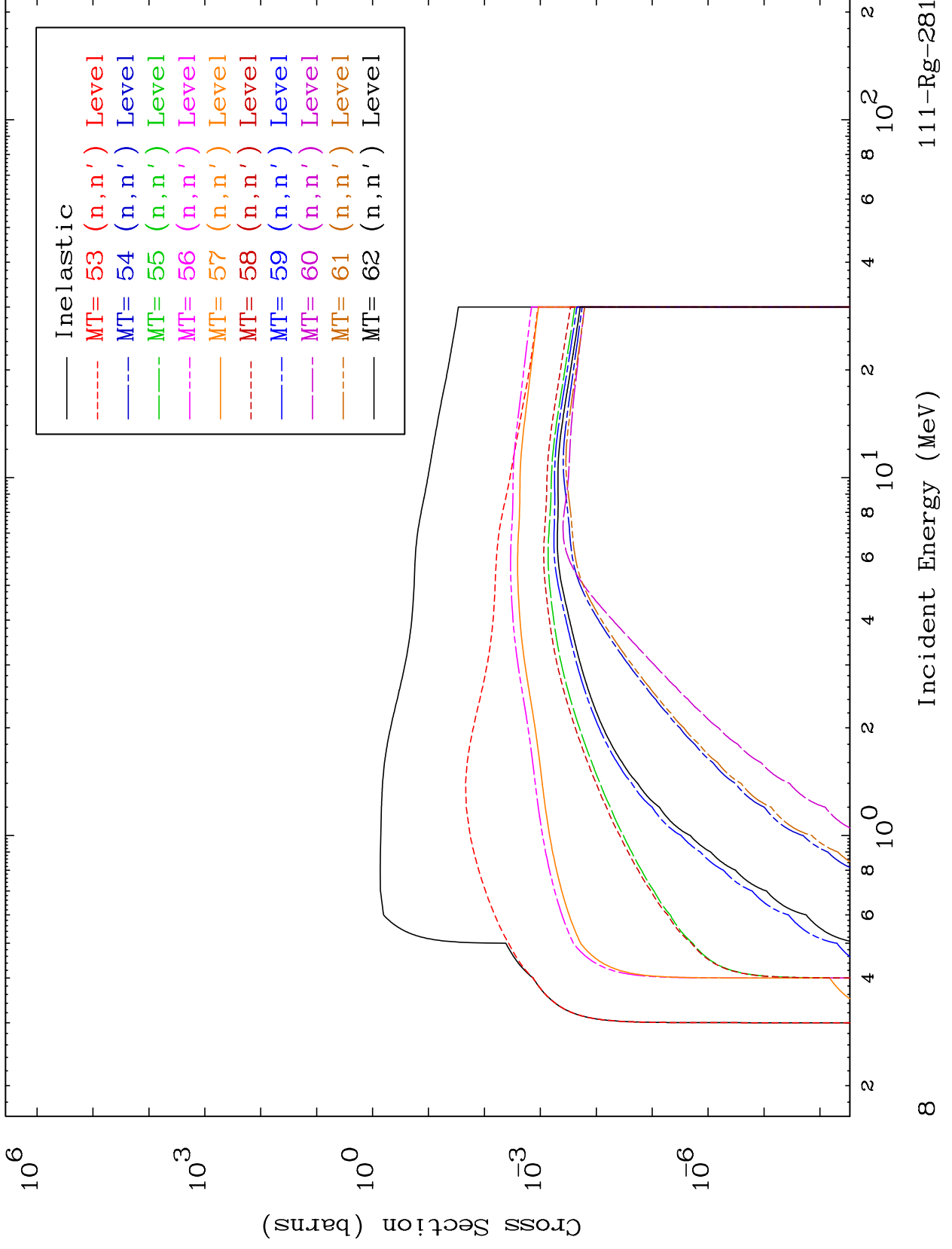




MAT 1181

(n,n') Levels  
293 Kelvin Cross Sections

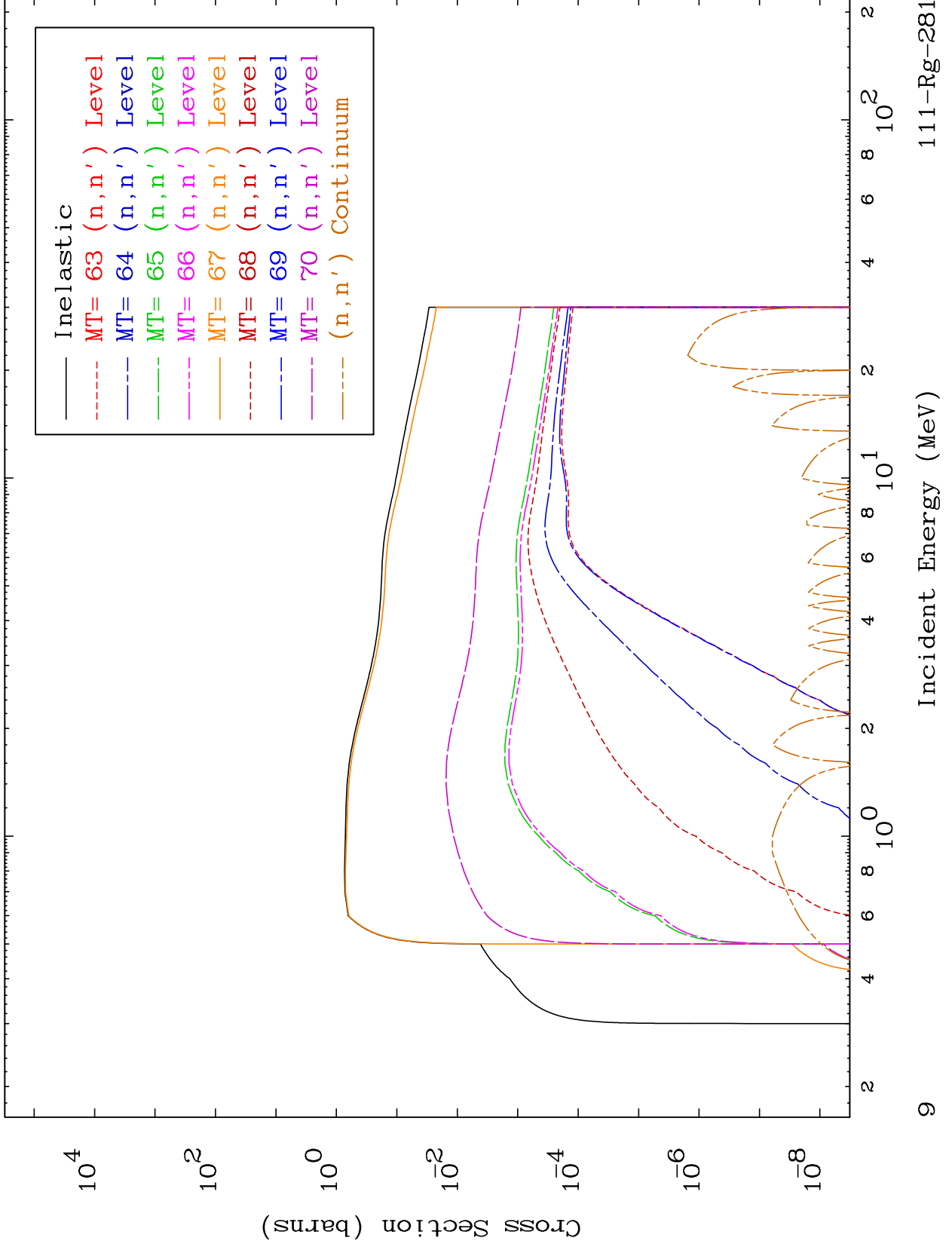
111-Rg-281



8

Incident Energy (MeV)

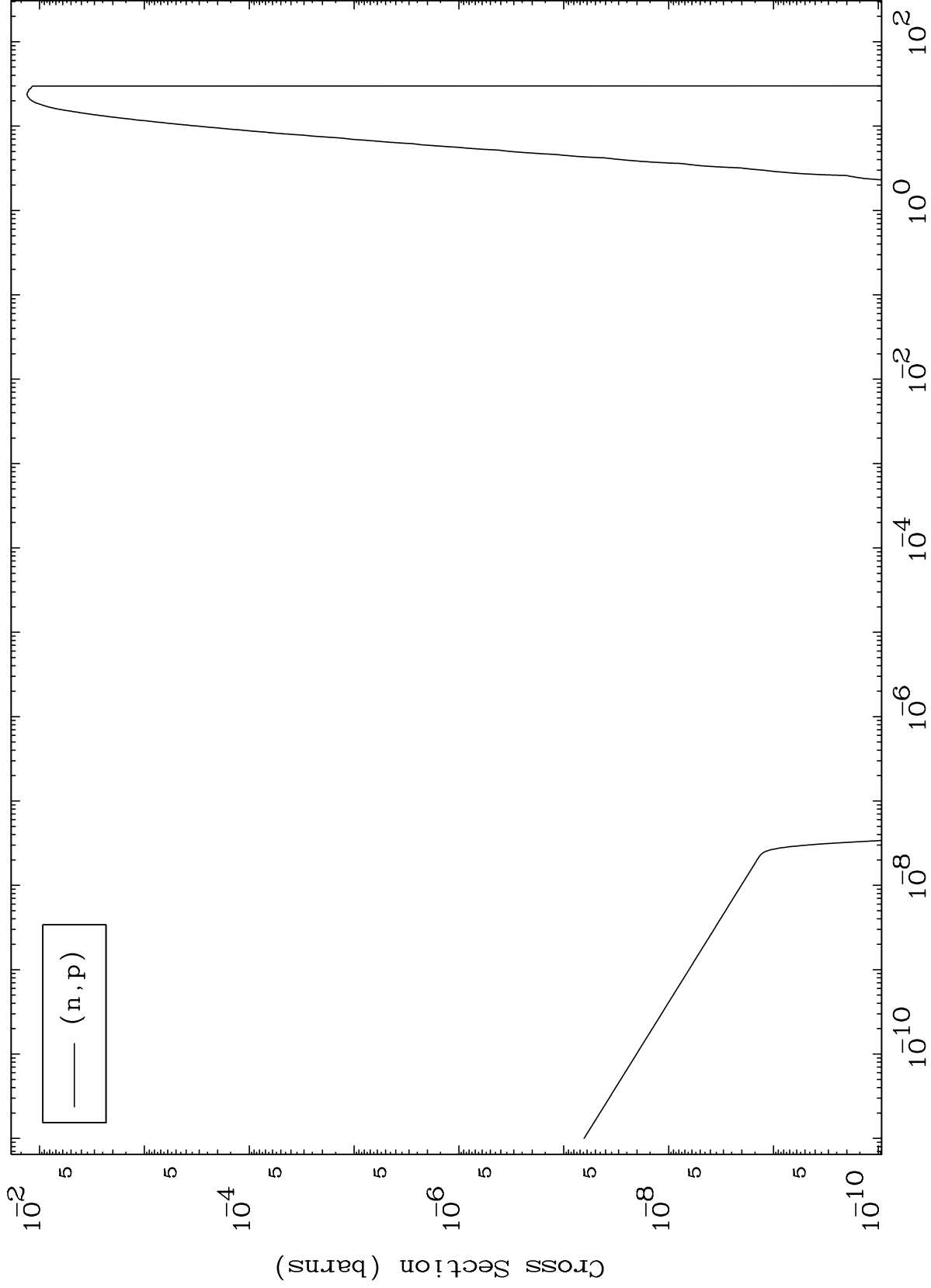
111-Rg-281



MAT 1181

(n,p) Levels  
293 Kelvin Cross Sections

111-Rg-281



10

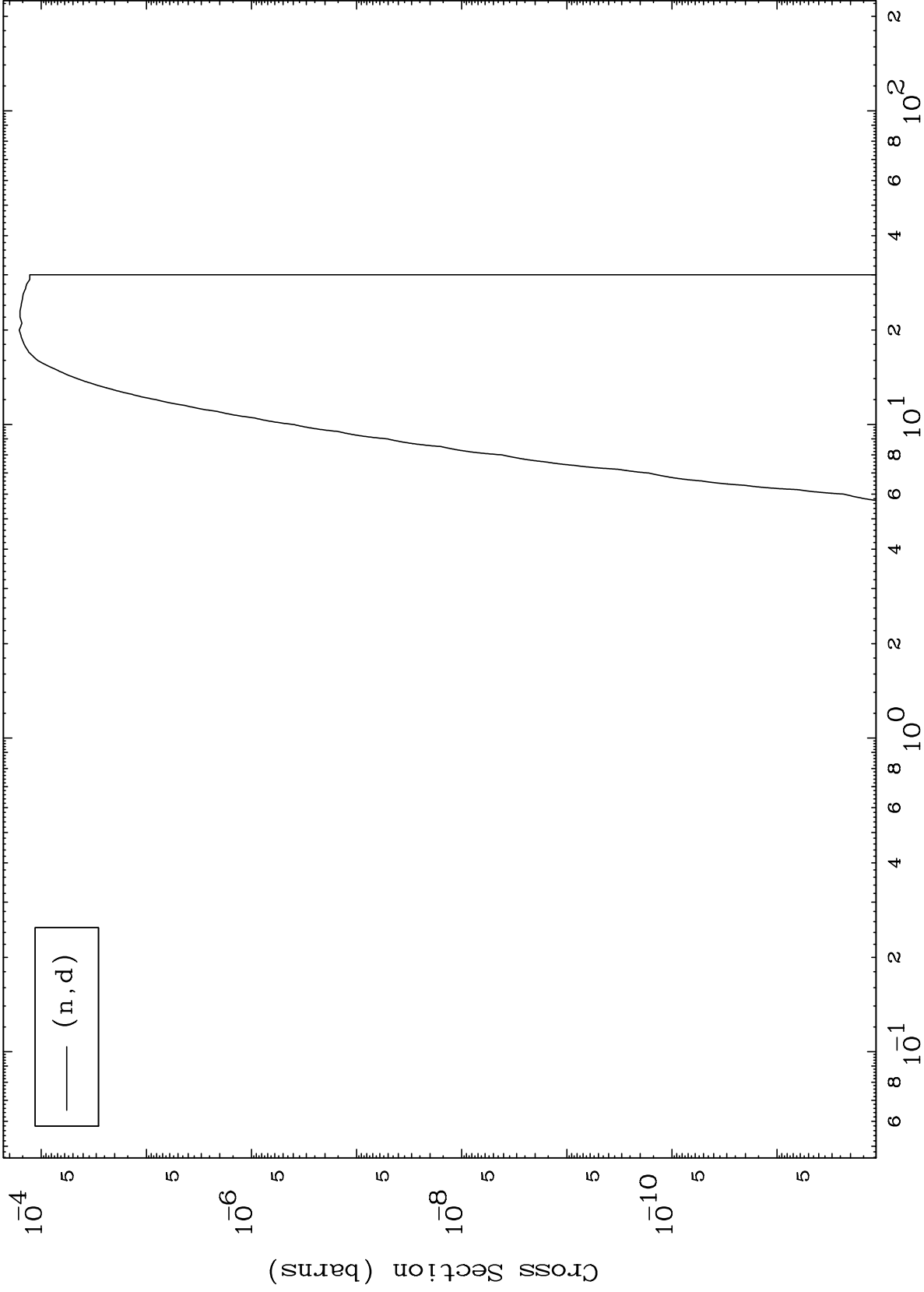
Incident Energy (MeV)

111-Rg-281

MAT 1181

(n,d) Levels  
293 Kelvin Cross Sections

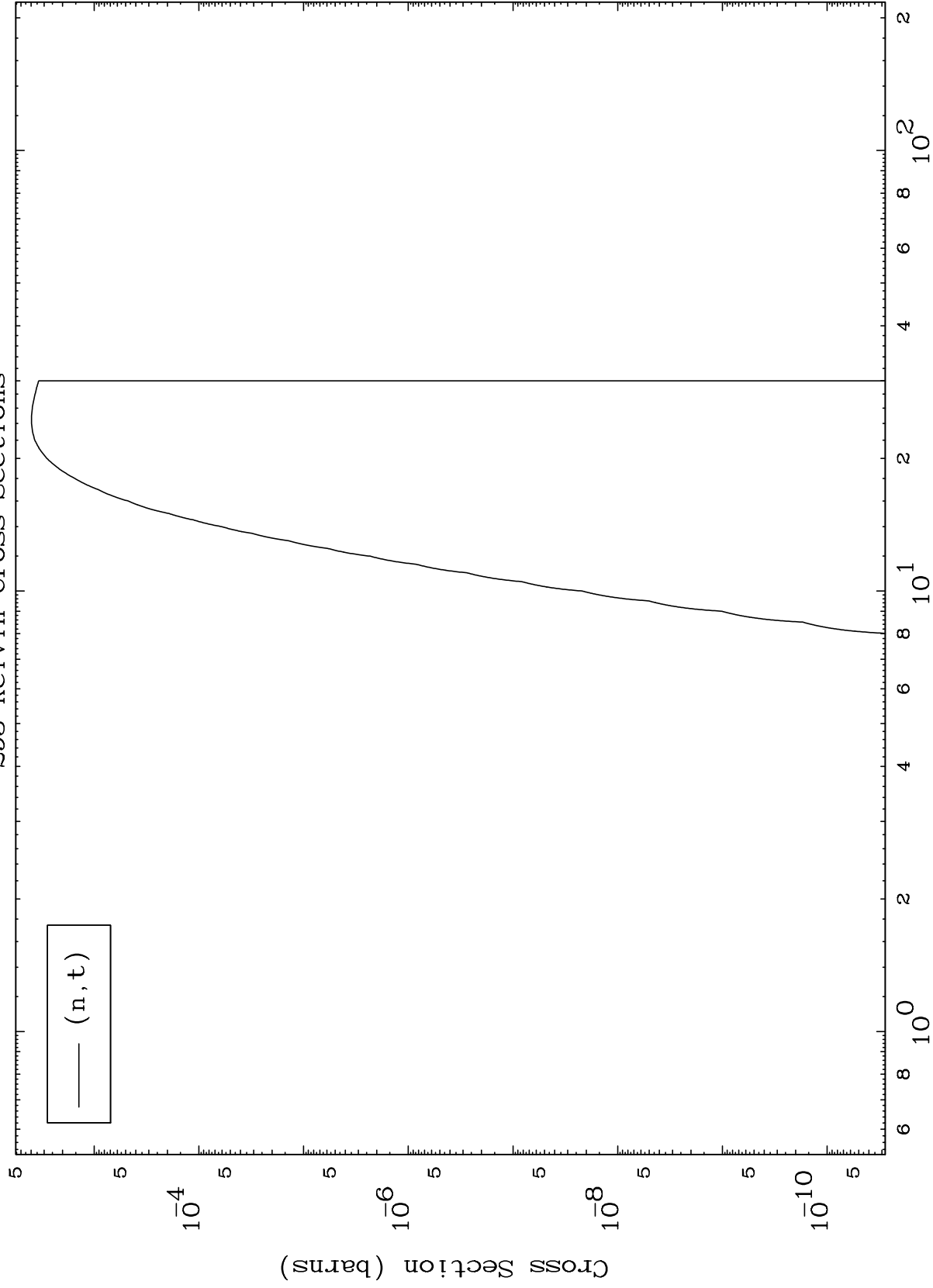
111-Rg-281



MAT 1181

(n,t) Levels  
293 Kelvin Cross Sections

111-Rg-281



12

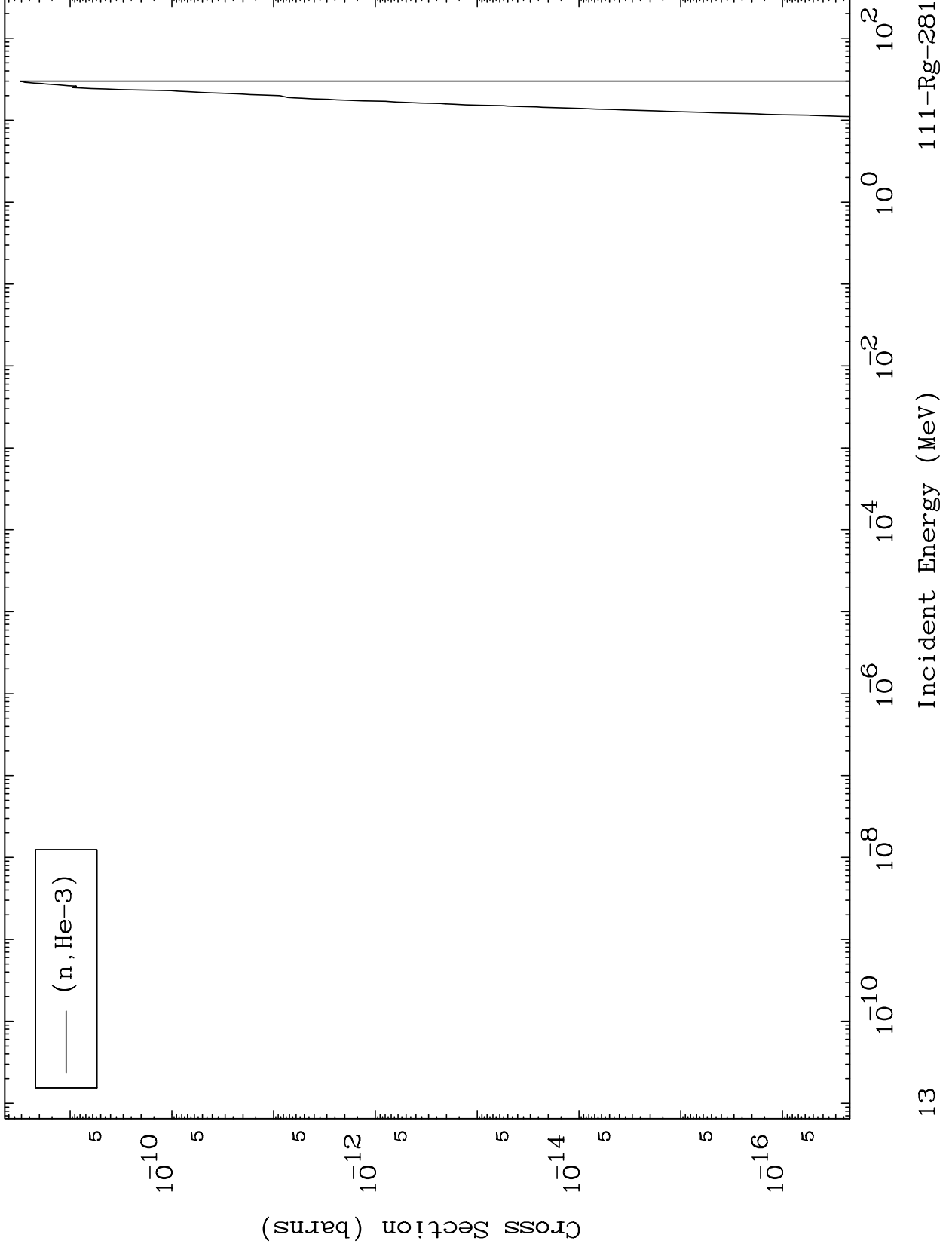
Incident Energy (MeV)

111-Rg-281

MAT 1181

(n,He3) Levels  
293 Kelvin Cross Sections

111-Rg-281



13

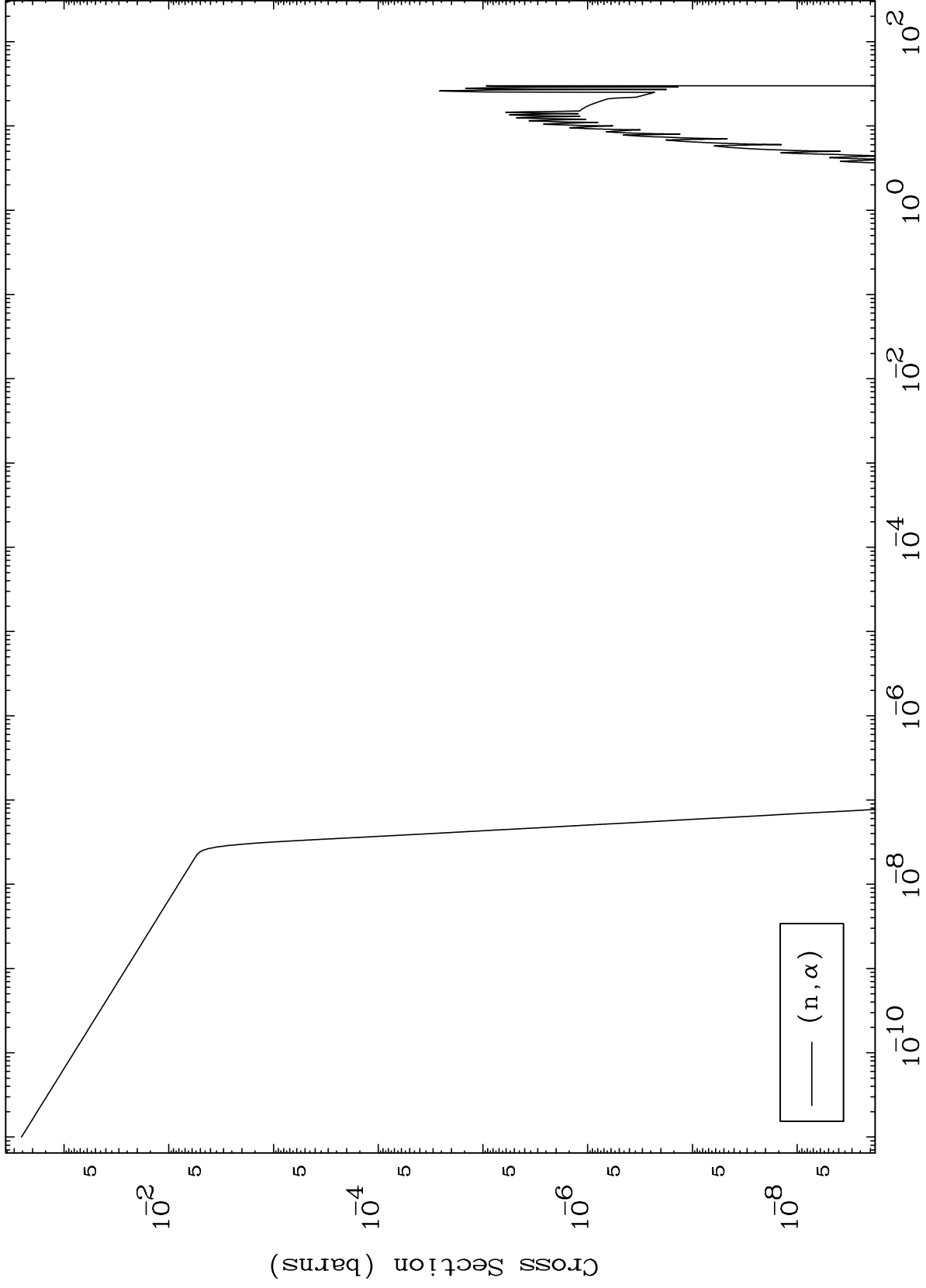
Incident Energy (MeV)

111-Rg-281

MAT 1181

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

111-Rg-281



14

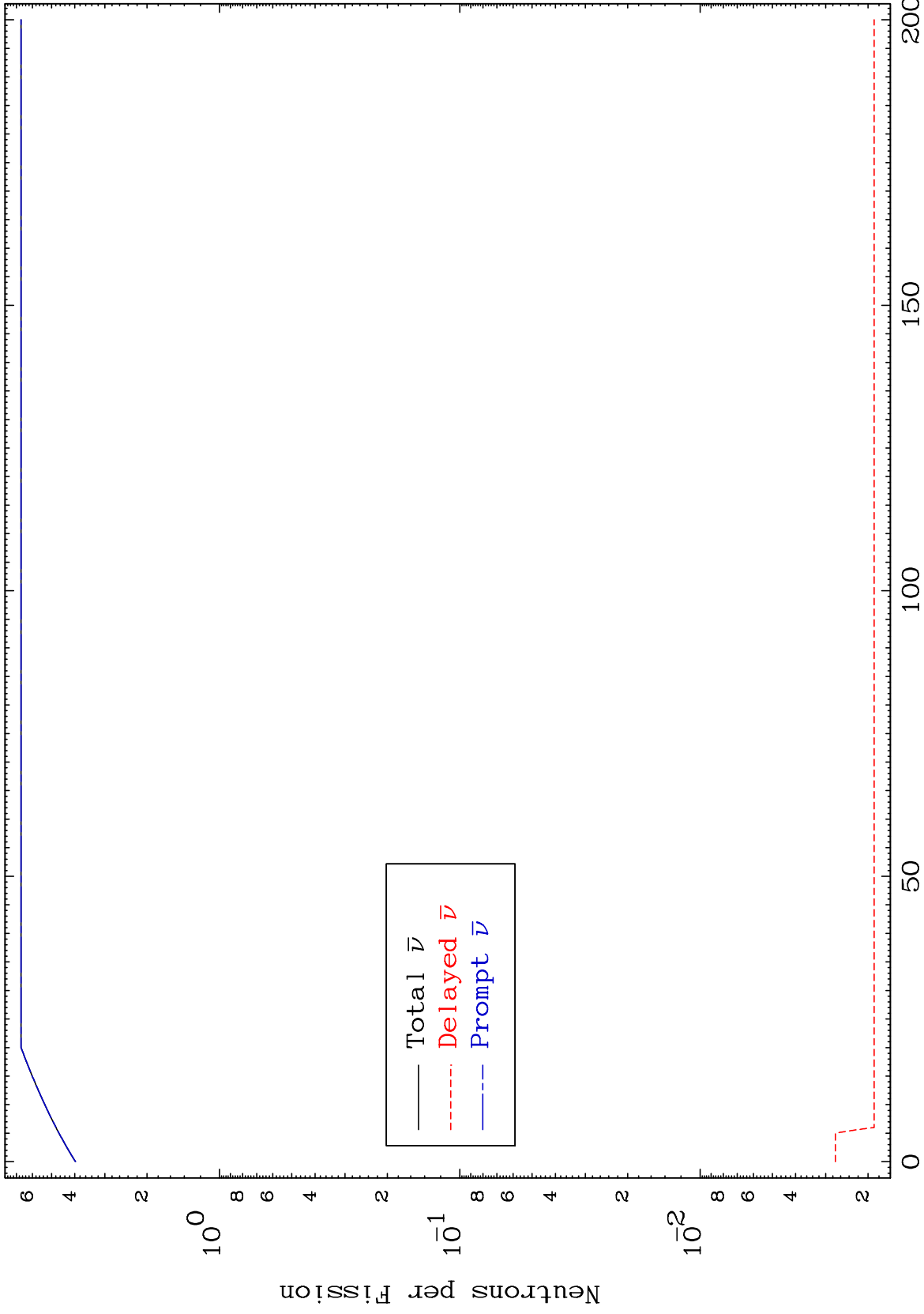
Incident Energy (MeV)

111-Rg-281

MAT 1181

111-Rg-281

Neutrons  
per Fission



0

50

100

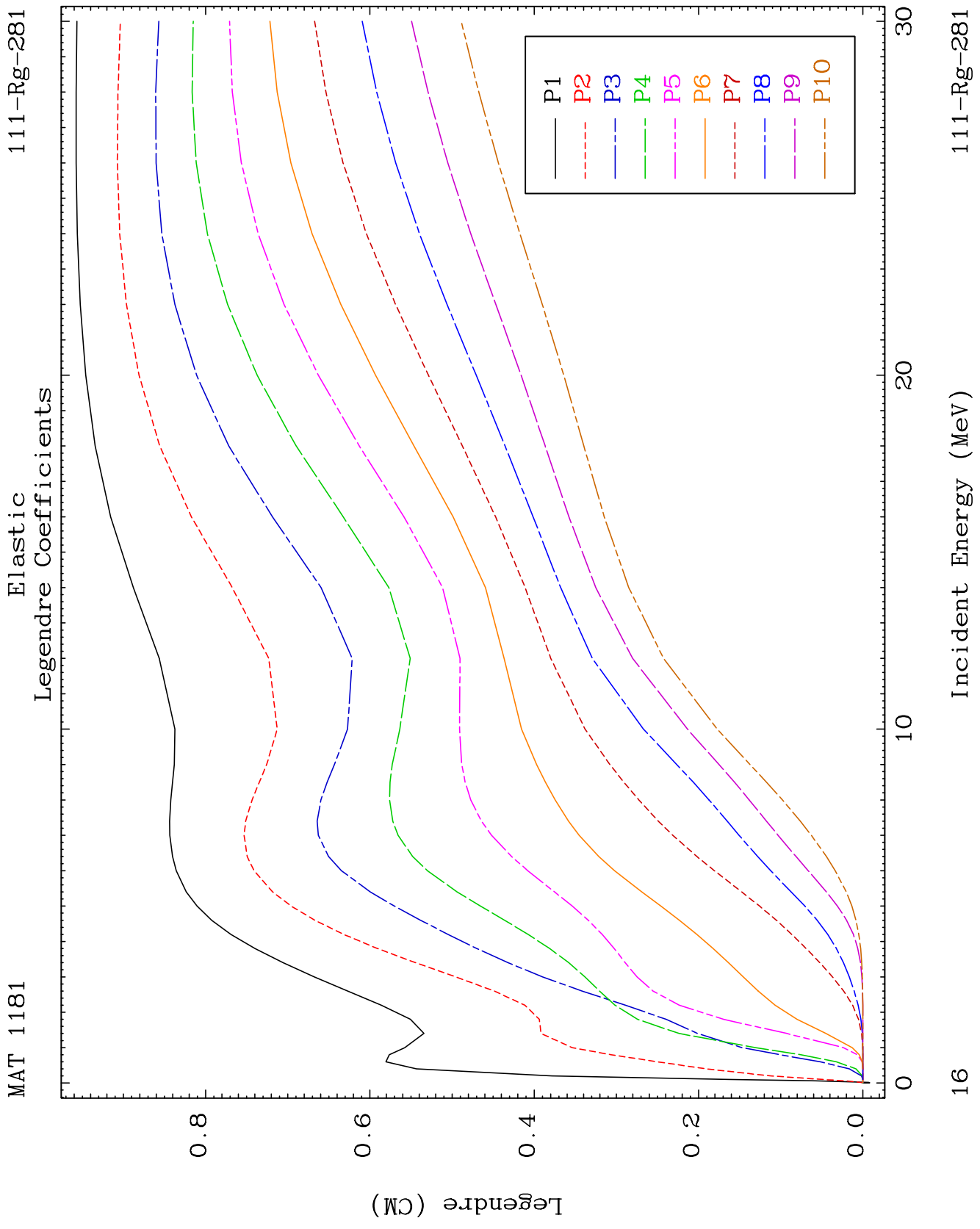
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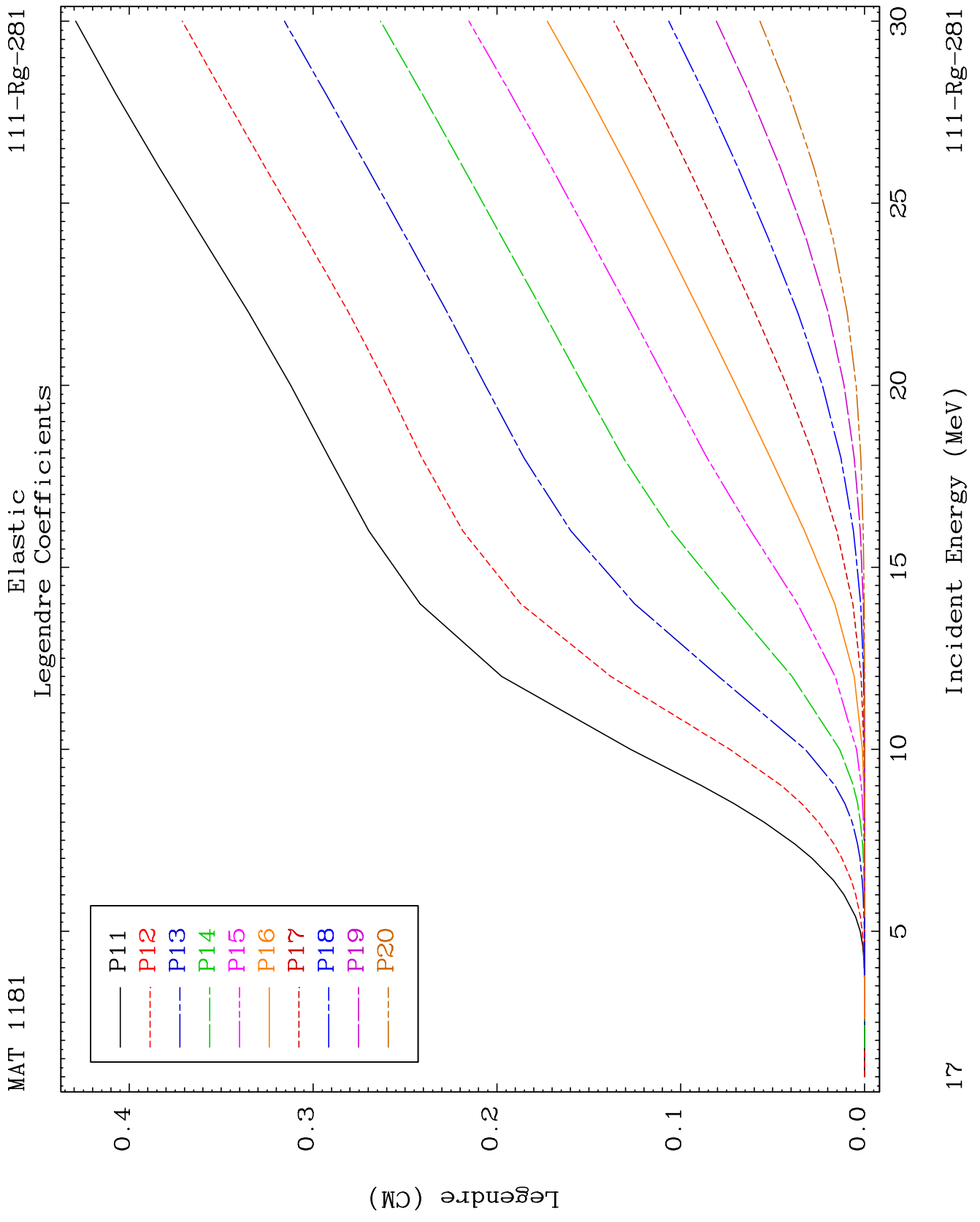
200

Incident Energy (MeV)

111-Rg-281



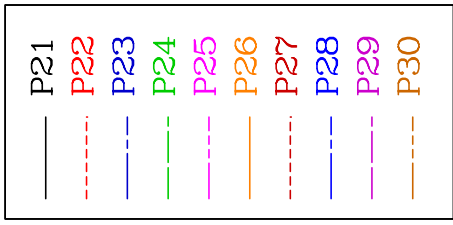
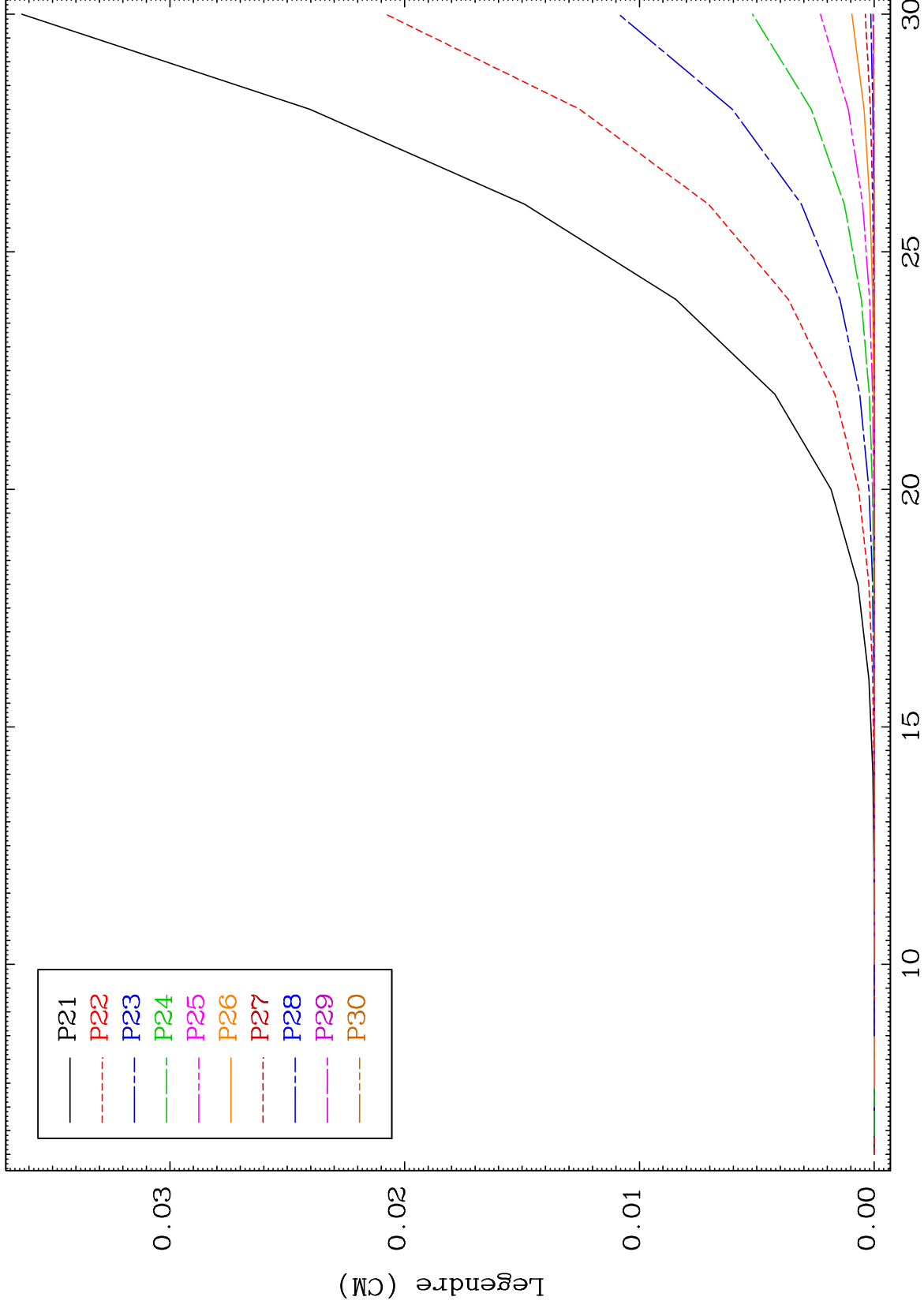




MAT 1181

Elastic  
Legendre Coefficients

111-Rg-281

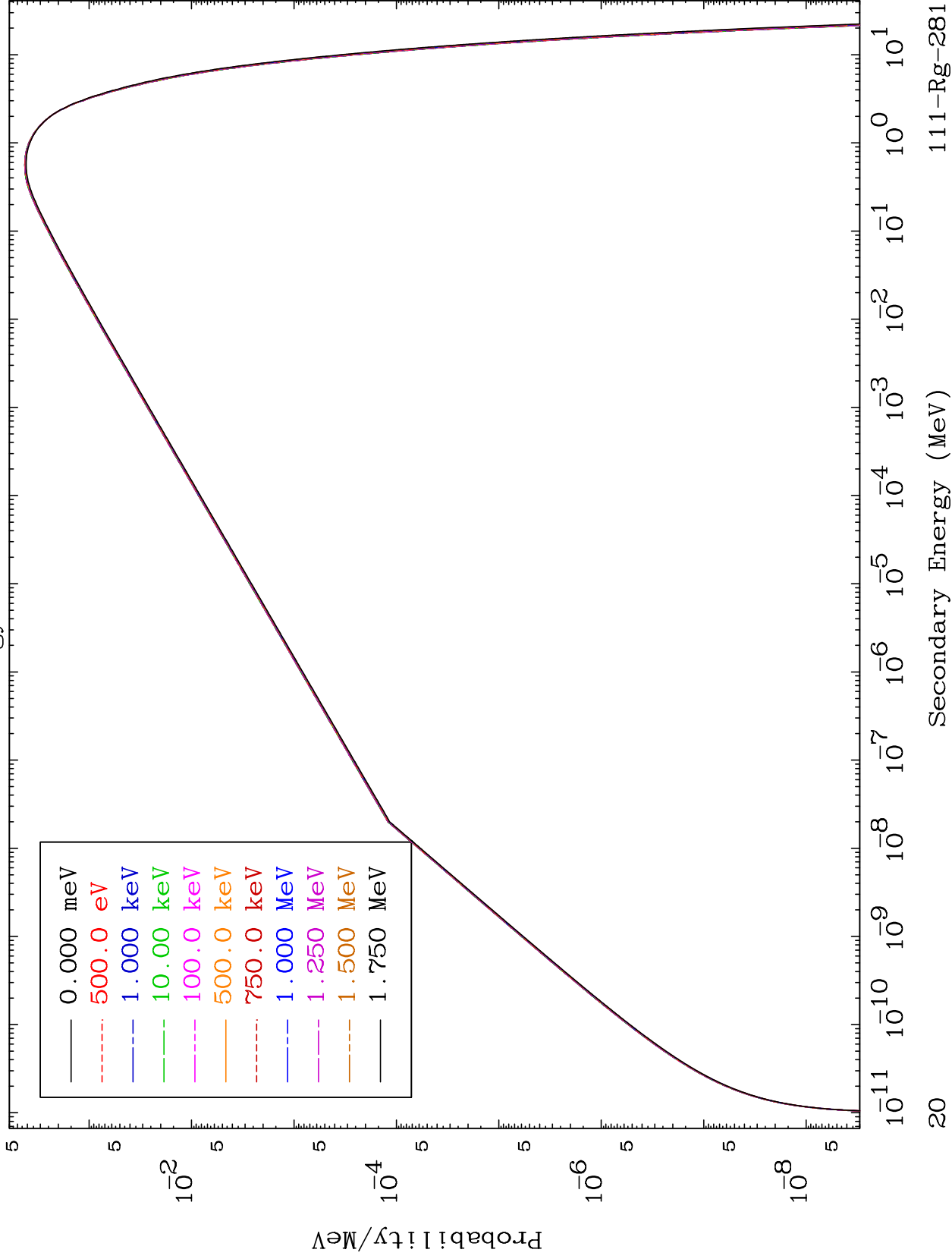




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Fission Energy Distributions

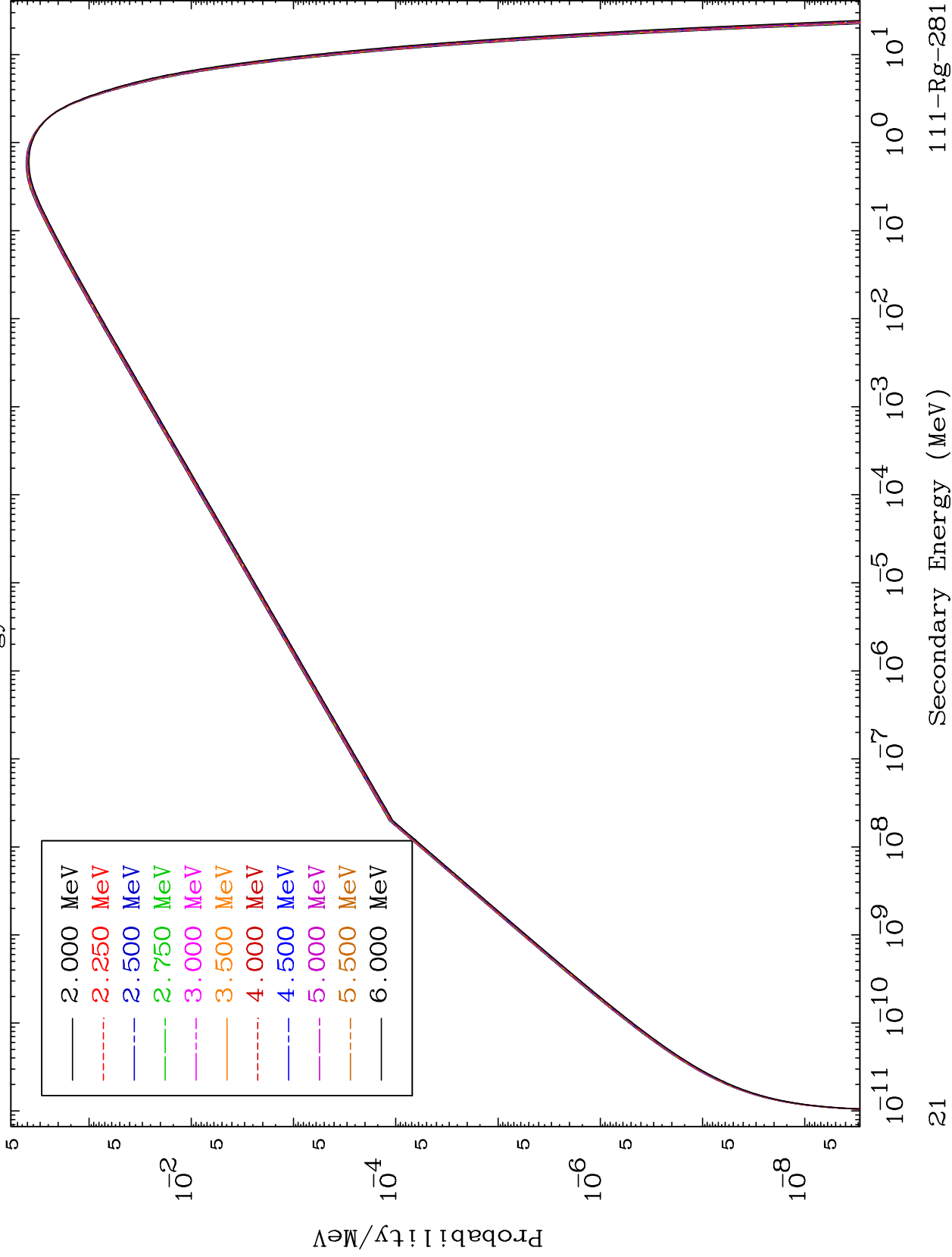
111-Rg-281



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### Fission Energy Distributions

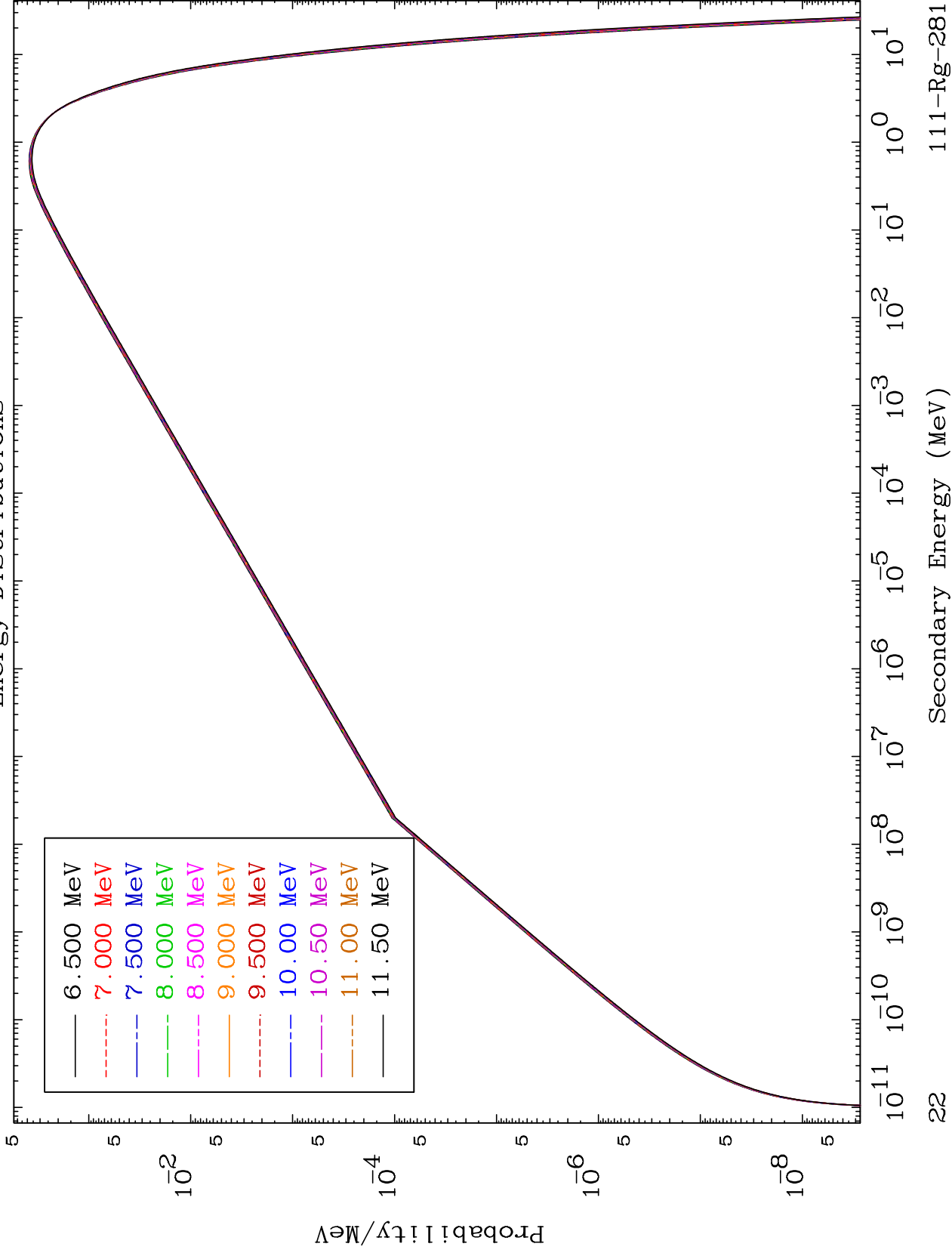
111-Rg-281



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Fission  
Energy Distributions

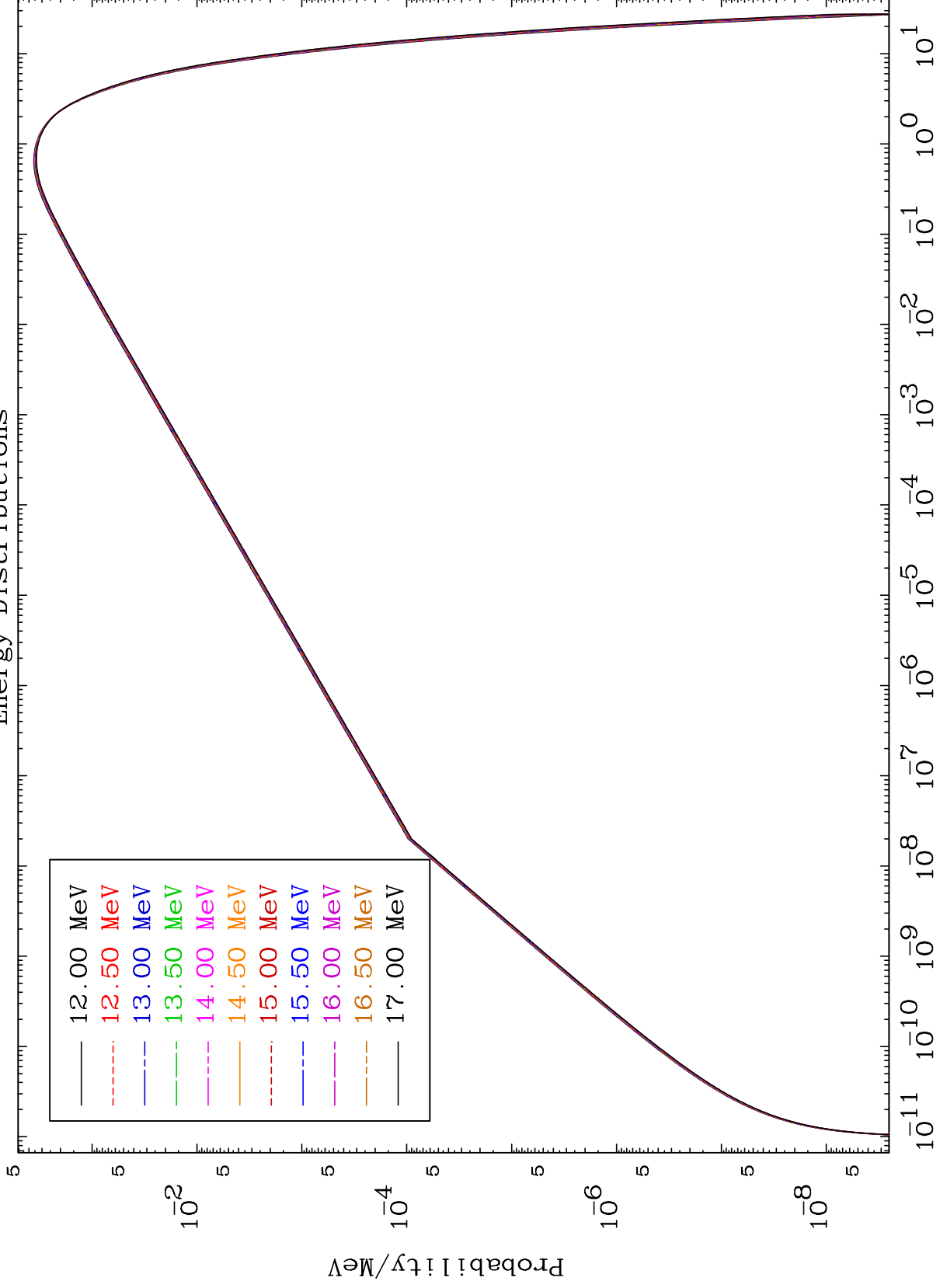
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Fission  
Energy Distributions

111-Rg-281



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Secondary Energy (MeV)

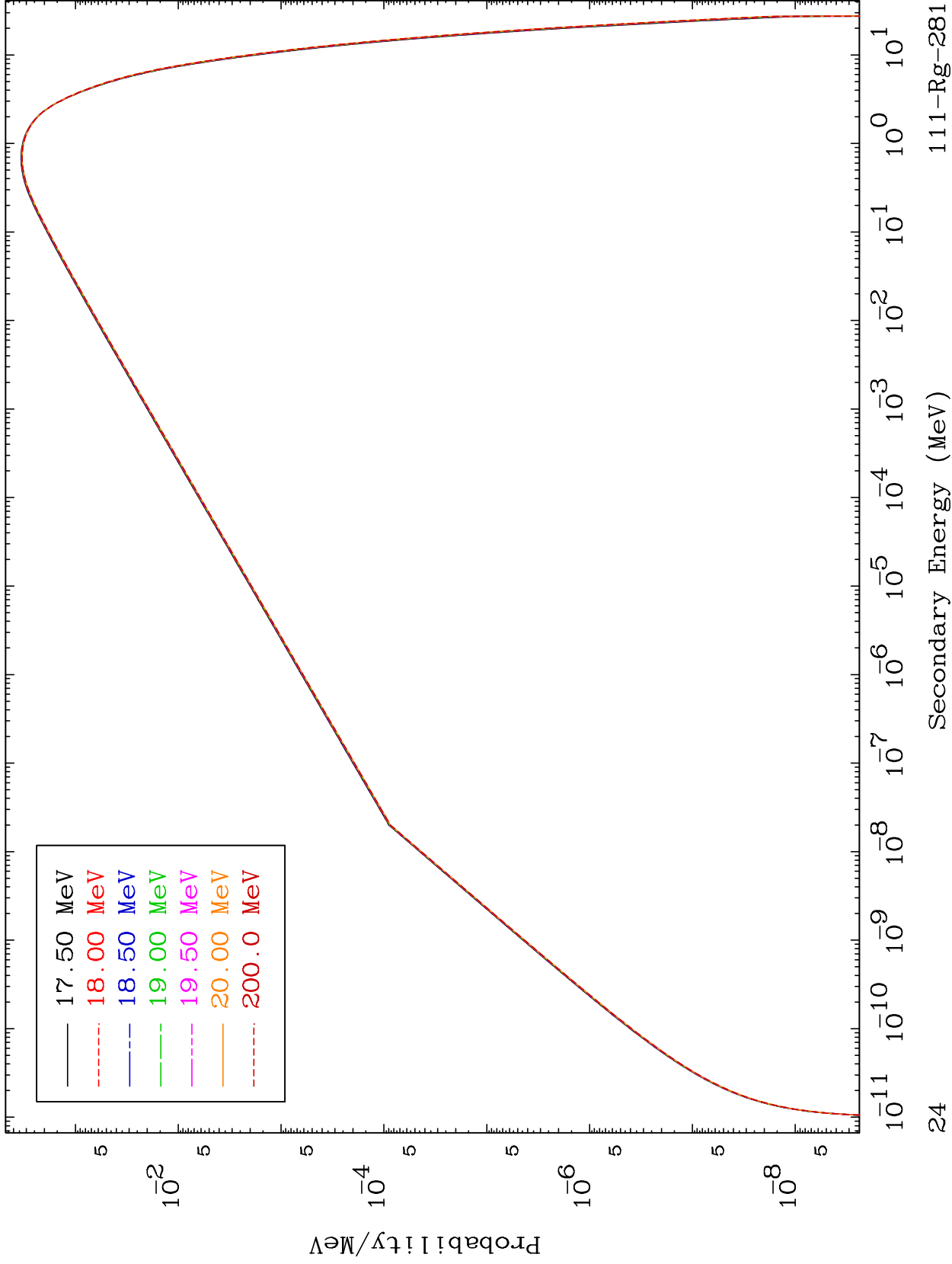
111-Rg-281



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### Fission Energy Distributions

111-Rg-281

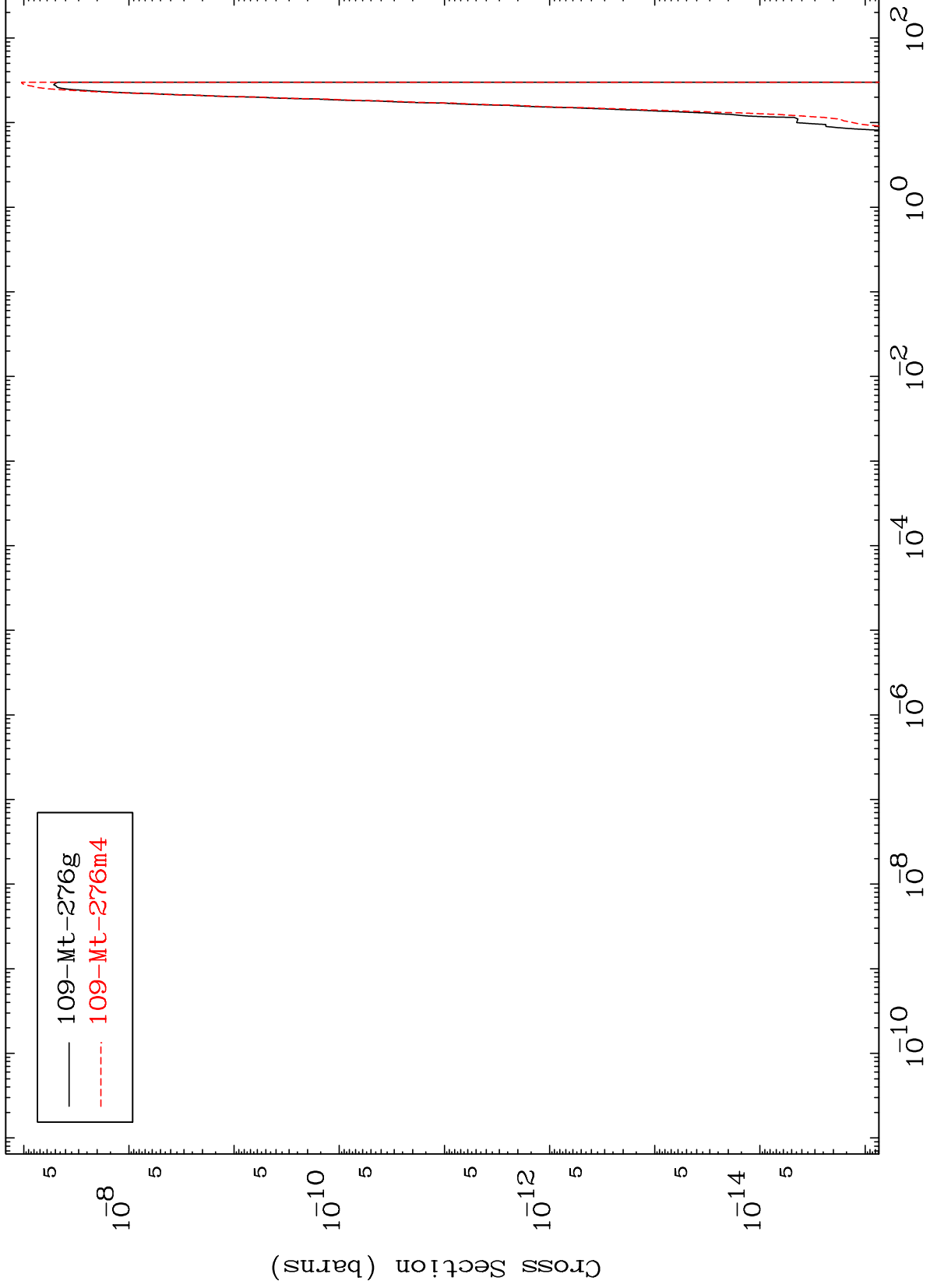


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(n,2n)  $\alpha$

111-Rg-281

Radionuclide Production Cross Section



25

Incident Energy (MeV)

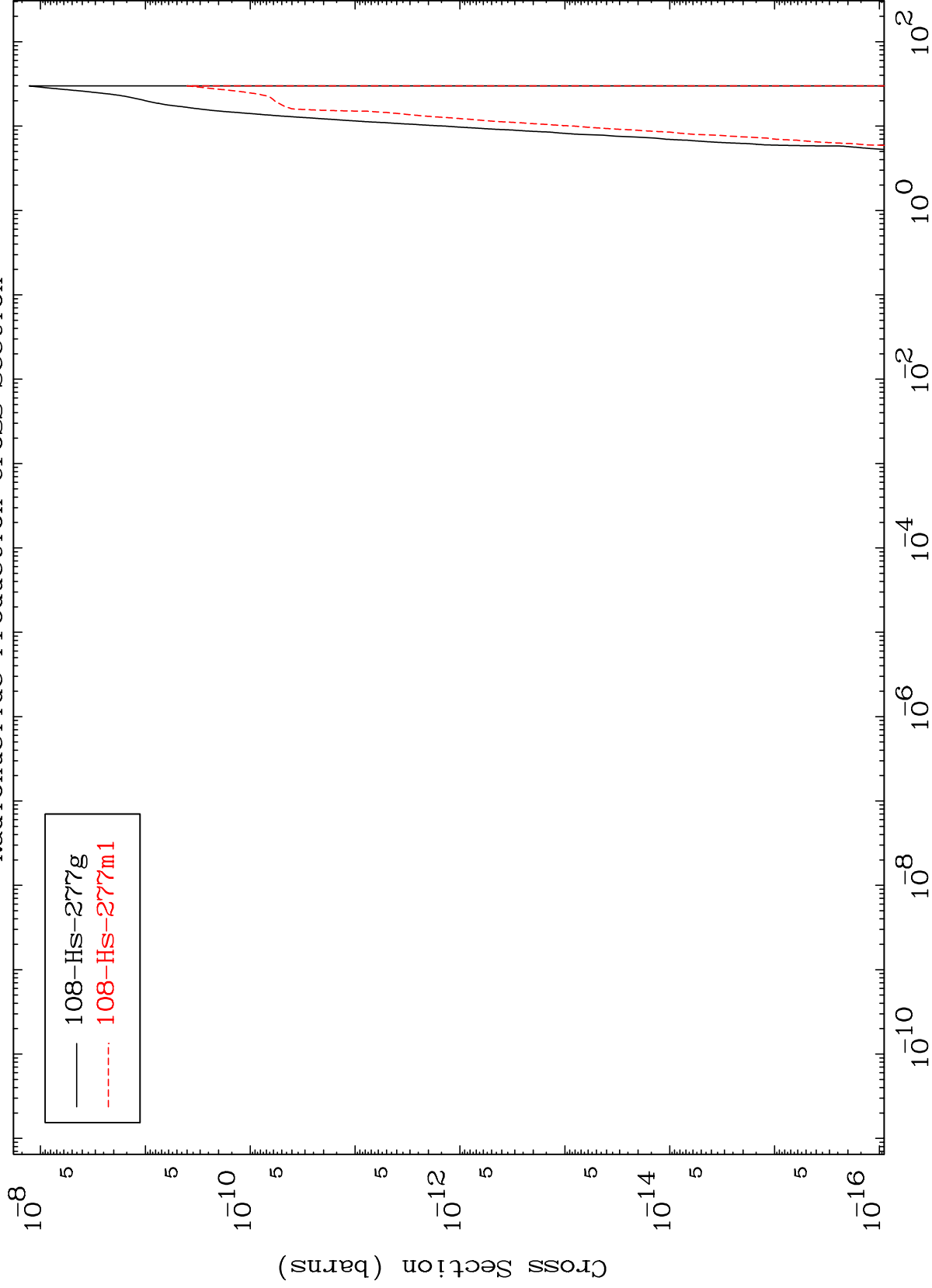
111-Rg-281

MAT 1181

(n,p)  $\alpha$

<sup>111</sup>Rg-281

Radionuclide Production Cross Section



26

Incident Energy (MeV)

<sup>111</sup>Rg-281