

Program EVALPLOT  
(Version 2021-1)

by

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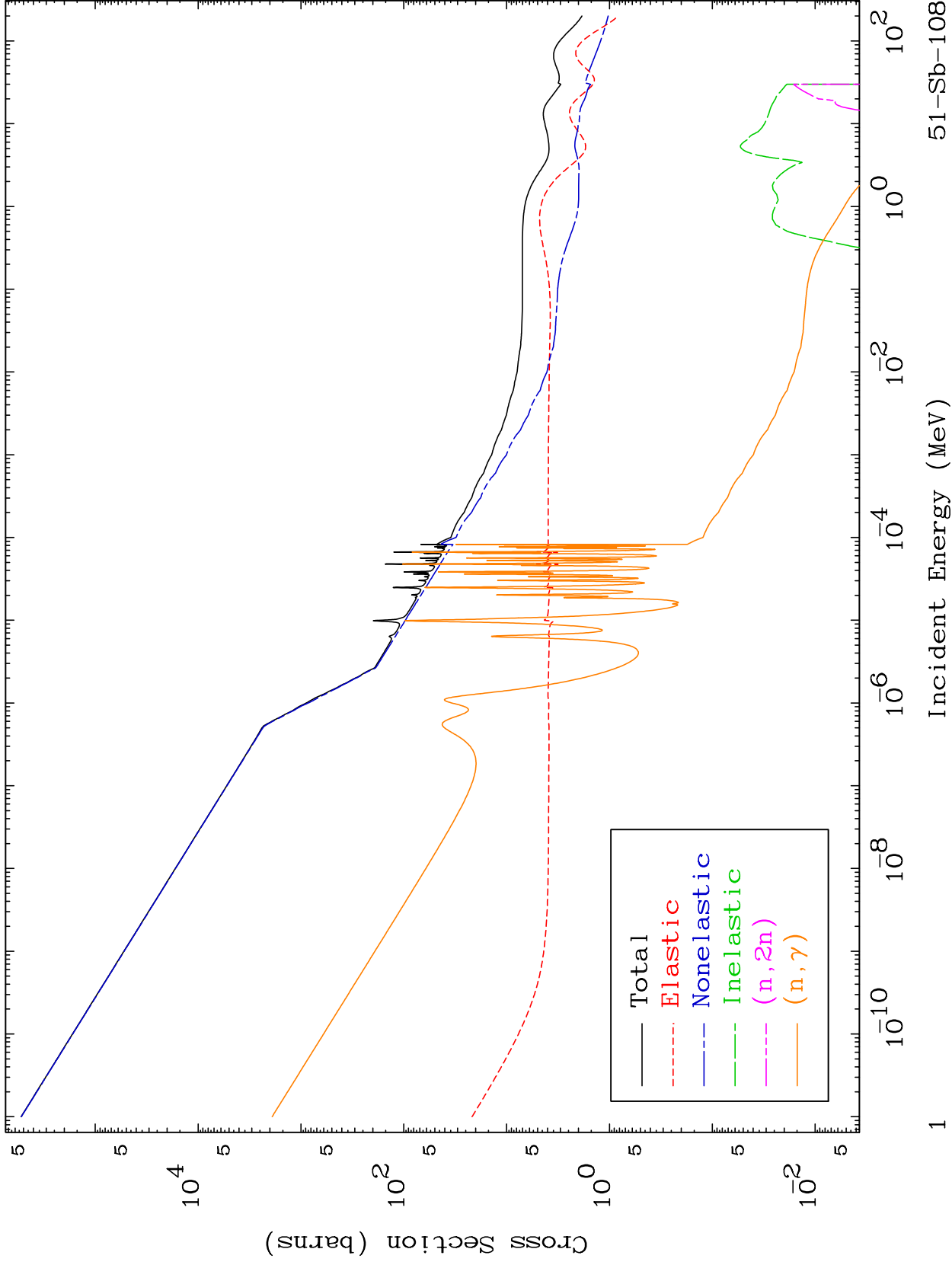
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 5086

Neutron Major  
293 Kelvin Cross Sections

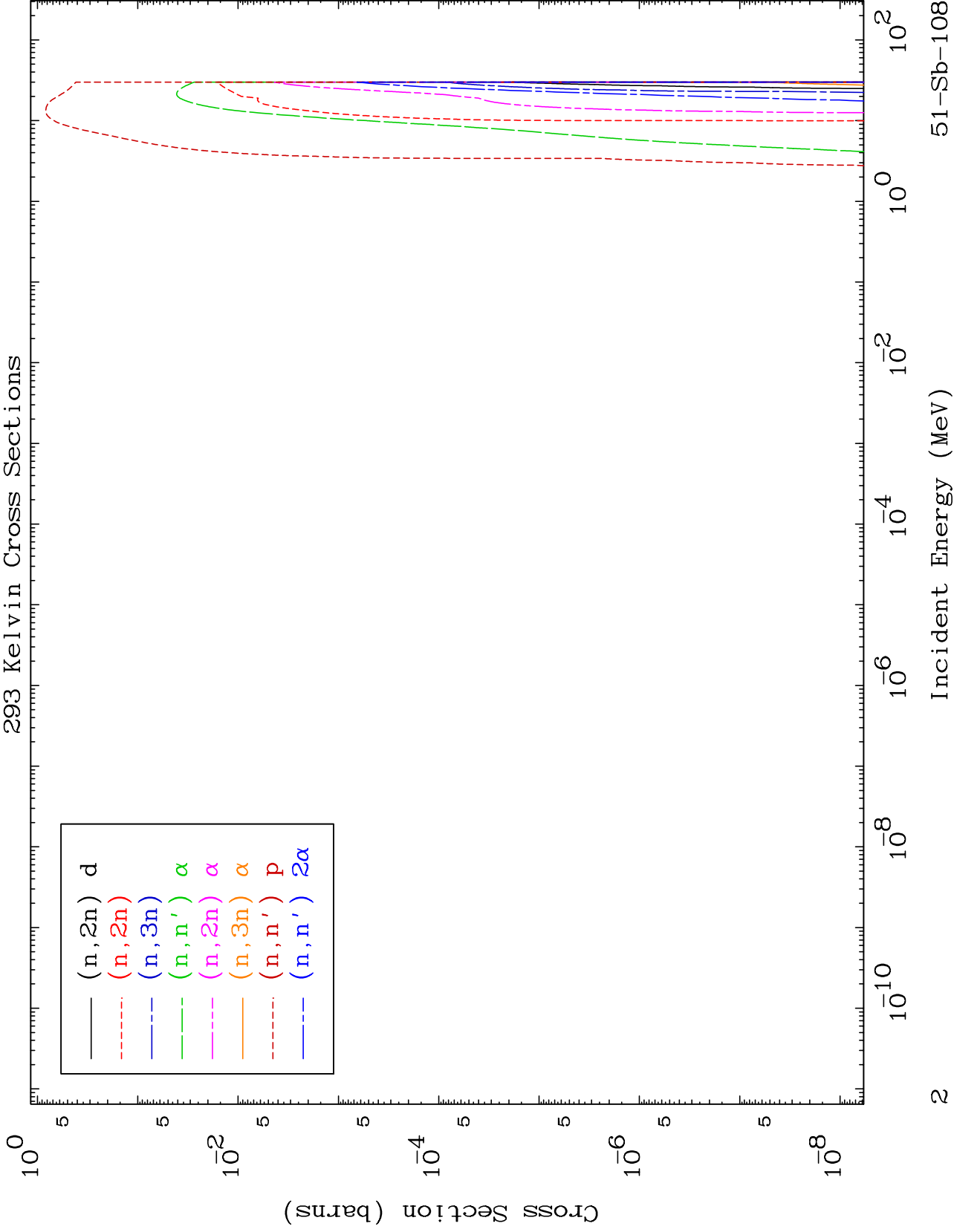
51-Sb-108

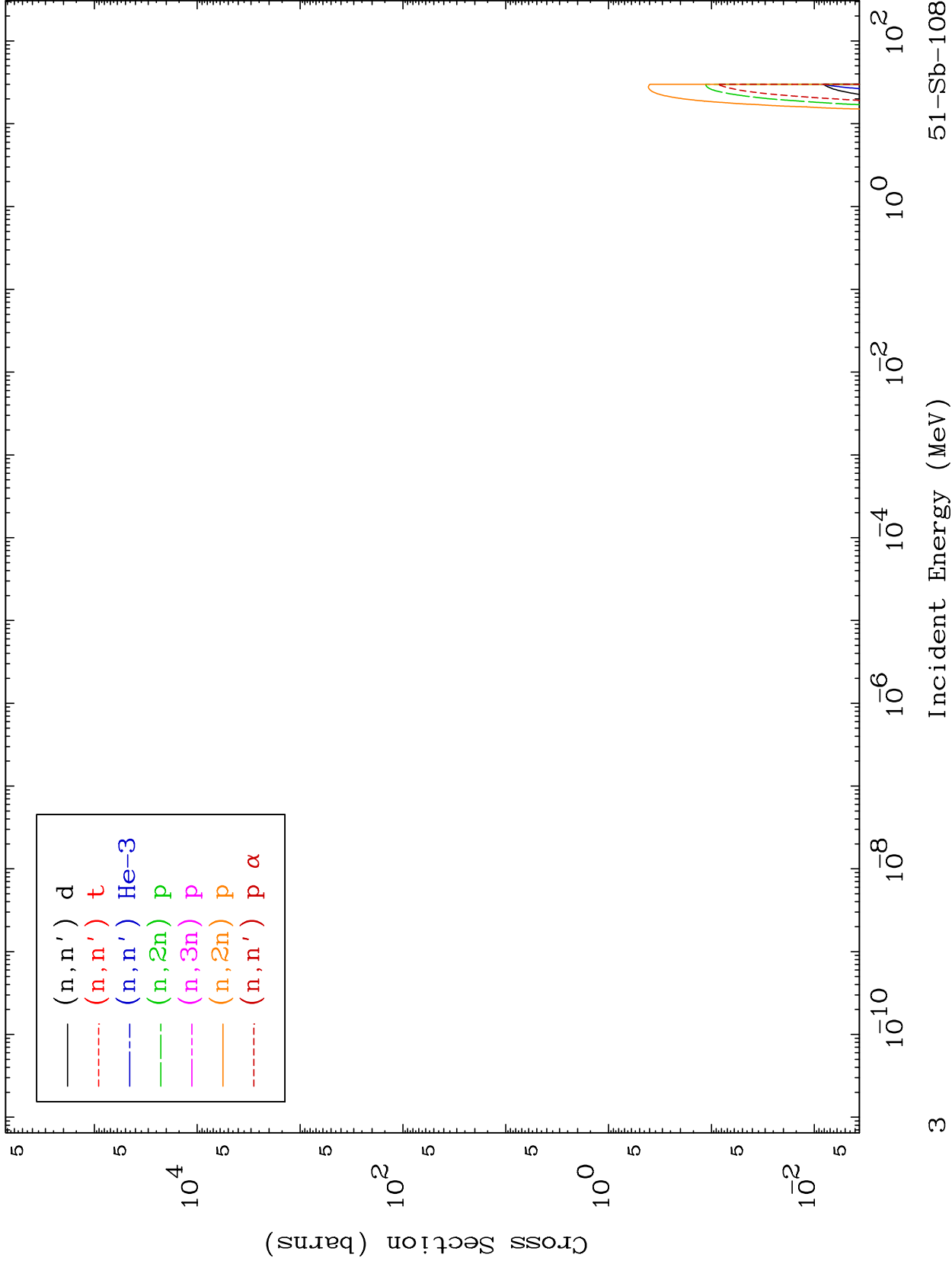


MAT 5086

Neutron Absorption  
293 Kelvin Cross Sections

51-Sb-108

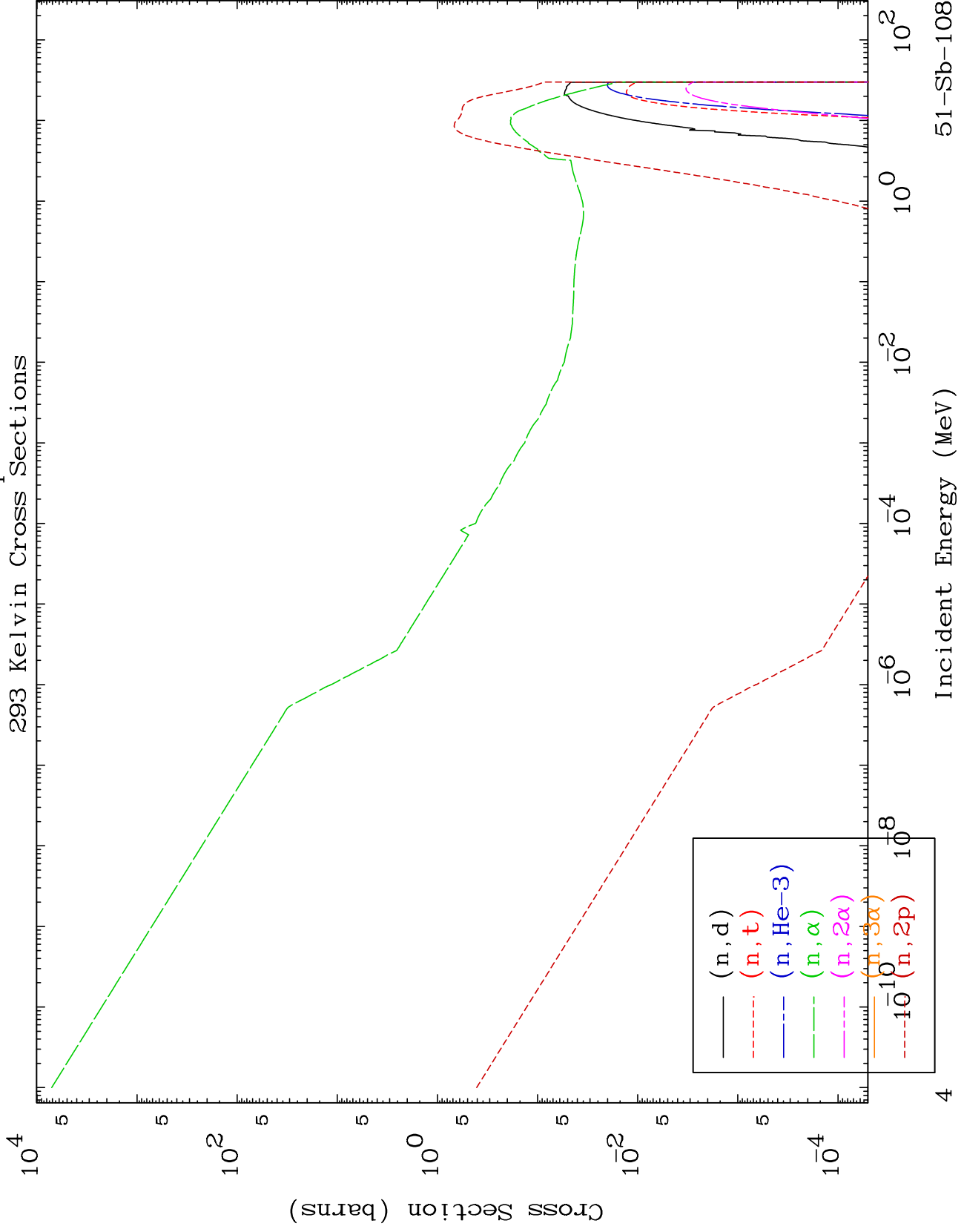


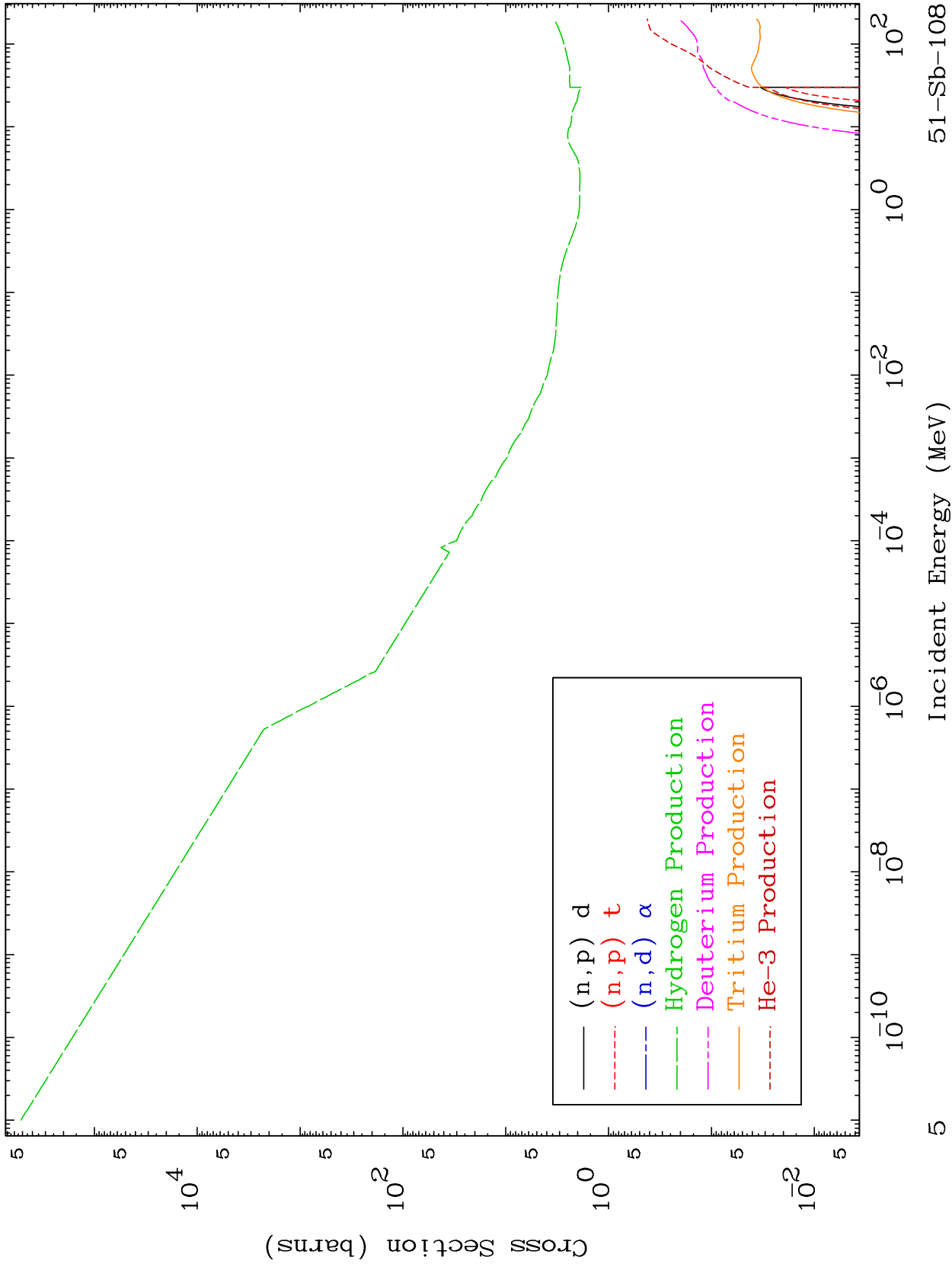


MAT 5086

Neutron Absorption  
293 Kelvin Cross Sections

51-Sb-108

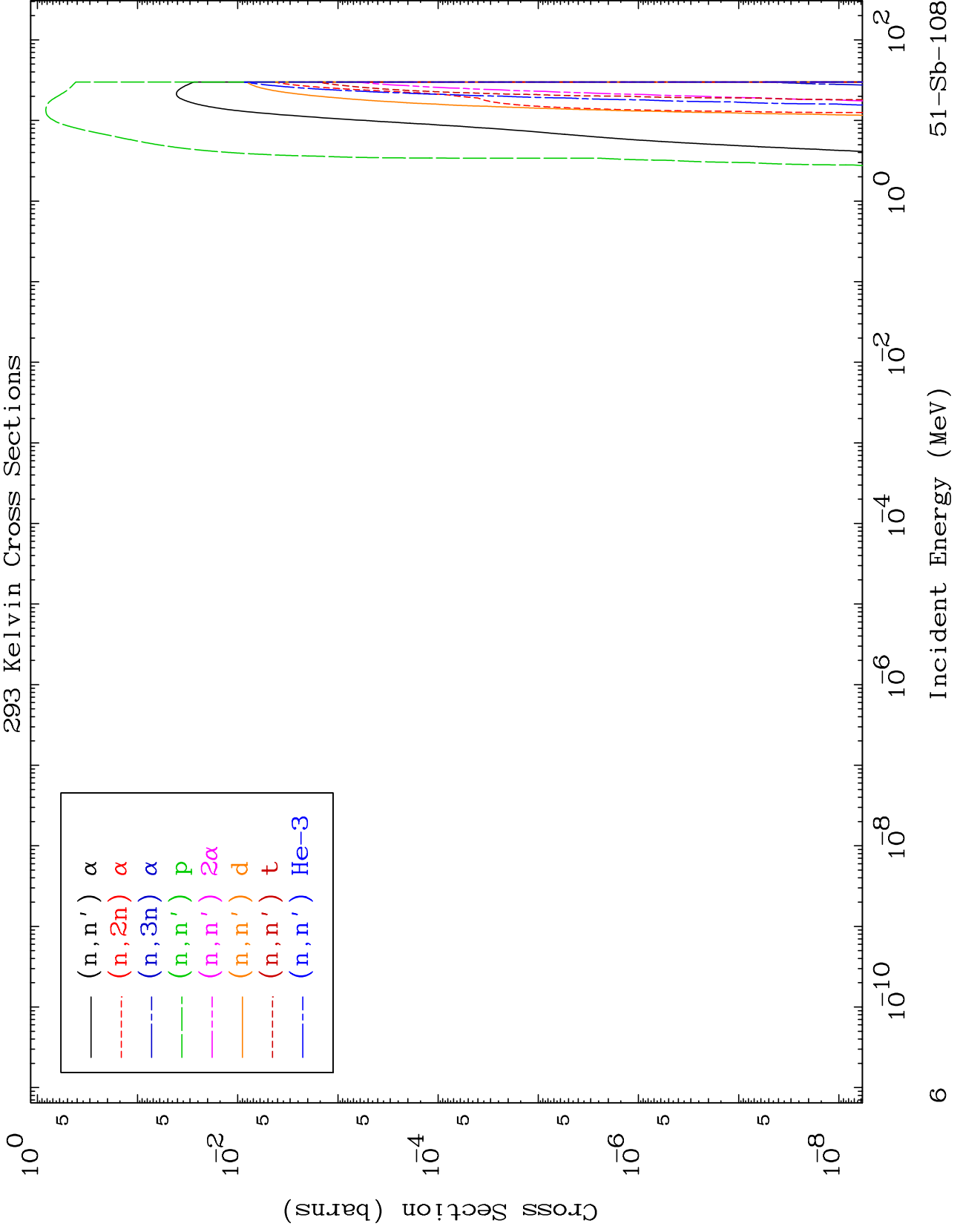




MAT 5086

Charged Particle  
293 Kelvin Cross Sections

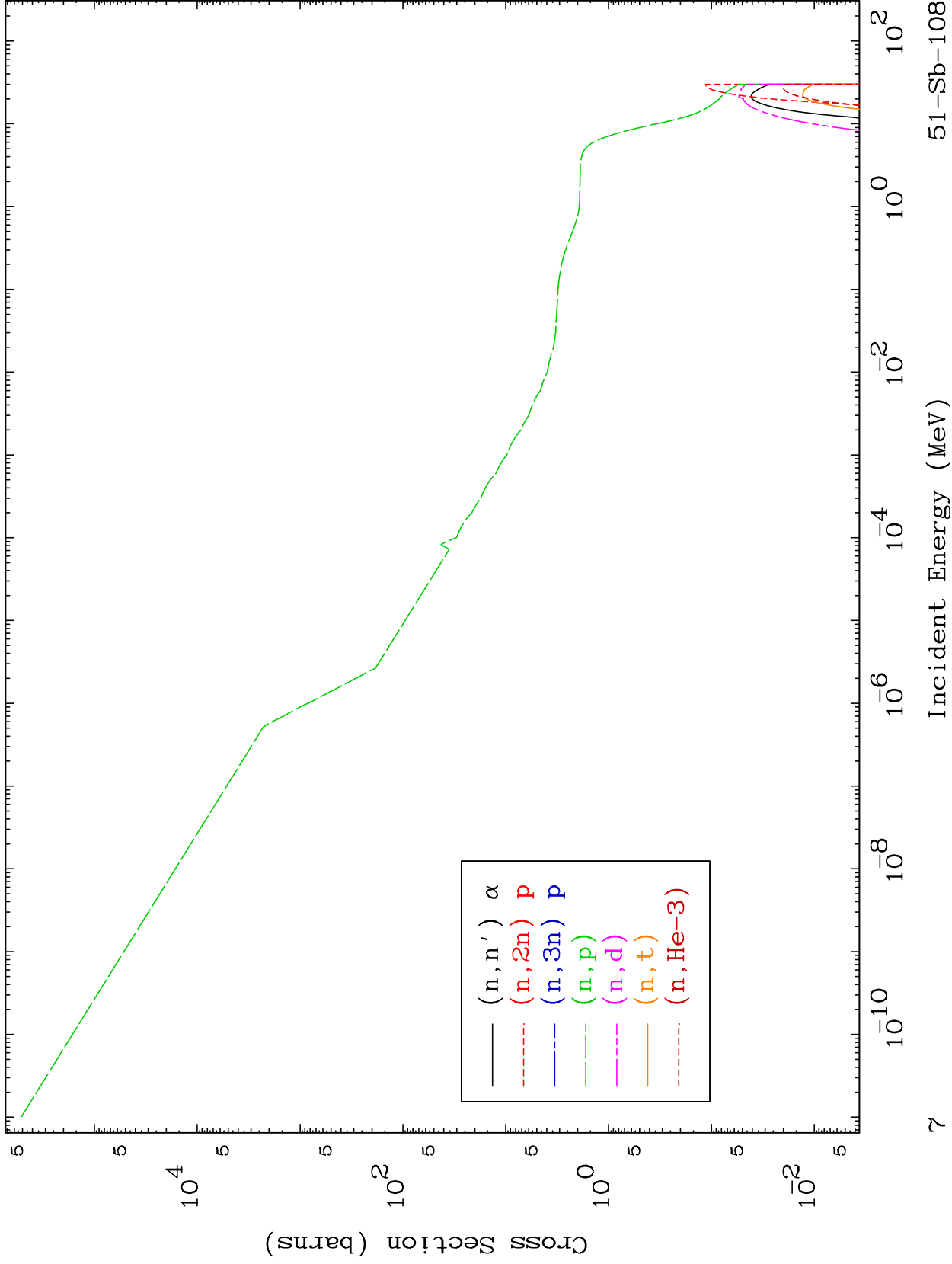
51-Sb-108



MAT 5086

Charged Particle  
293 Kelvin Cross Sections

51-Sb-108

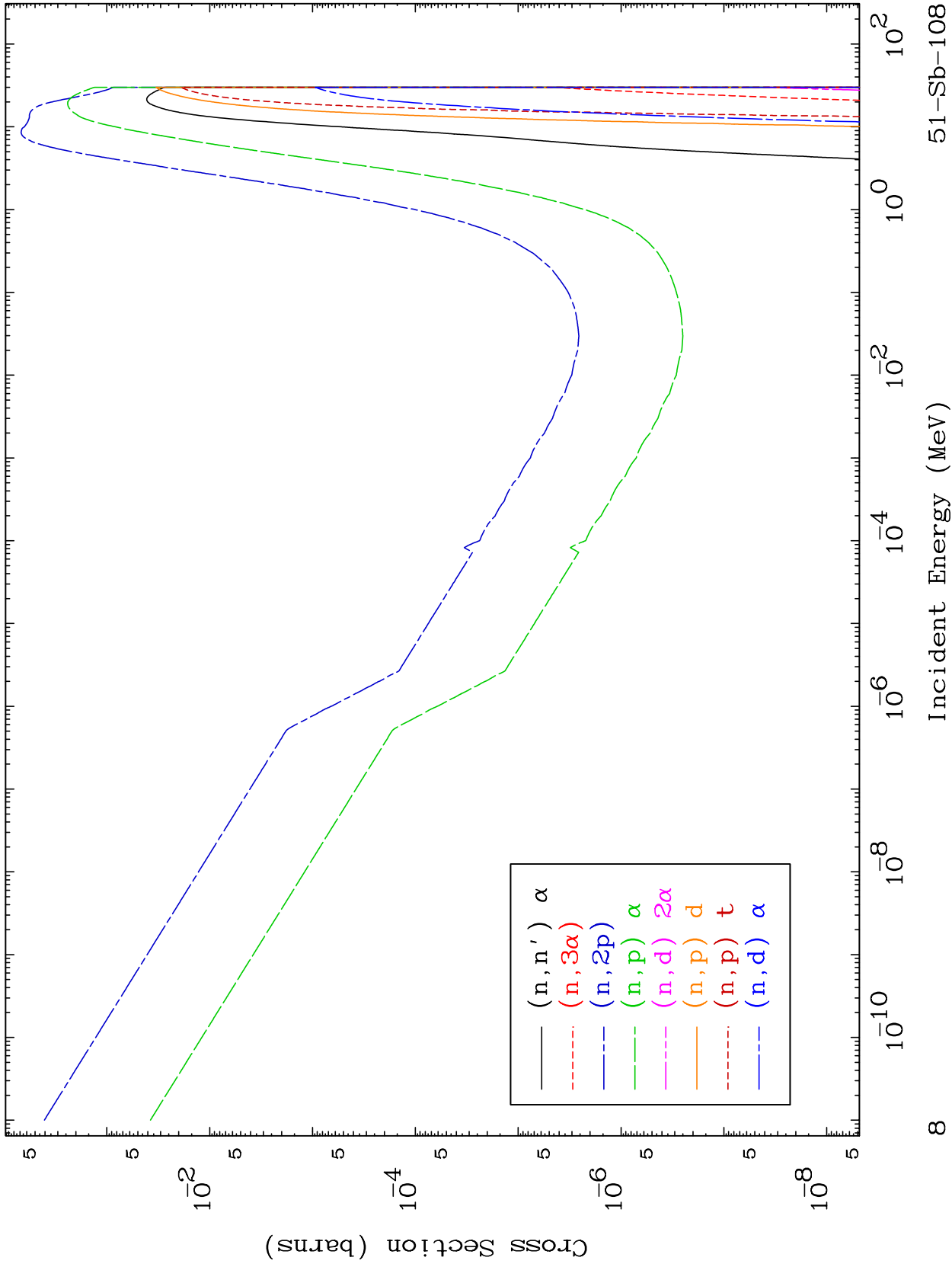


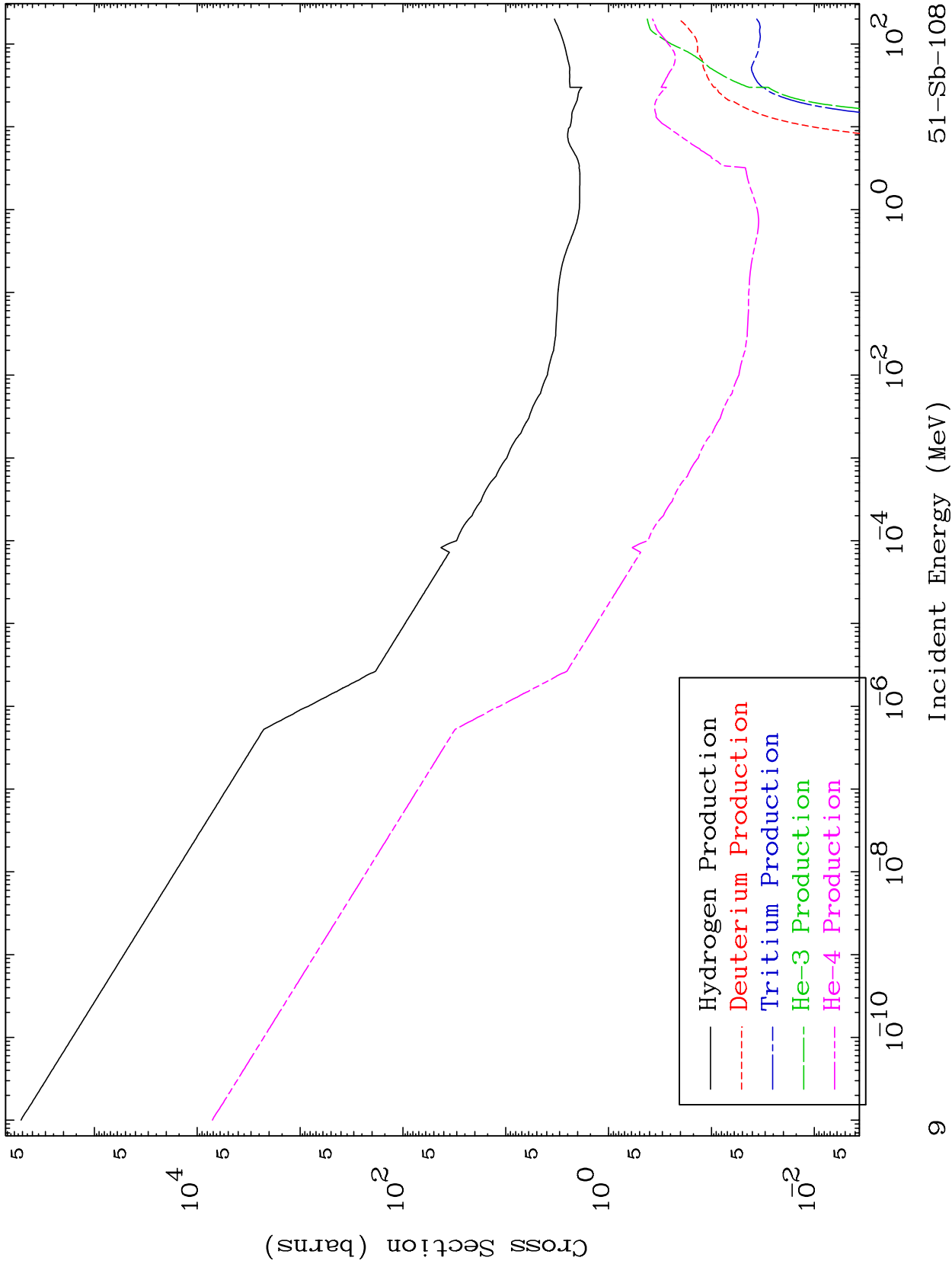


MAT 5086

Charged Particle  
293 Kelvin Cross Sections

51-Sb-108

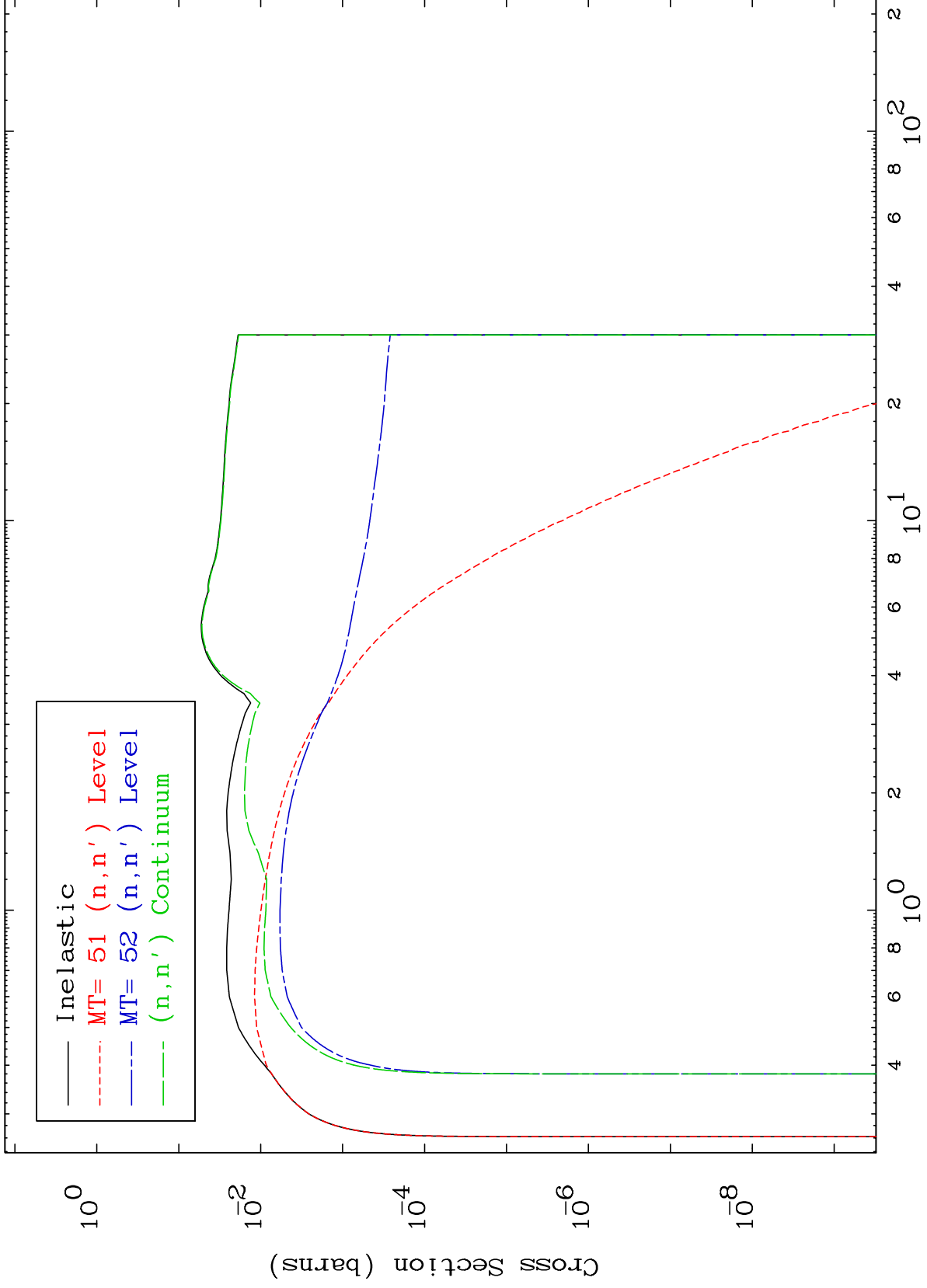




MAT 5086

(n,n') Levels  
293 Kelvin Cross Sections

51-Sb-108



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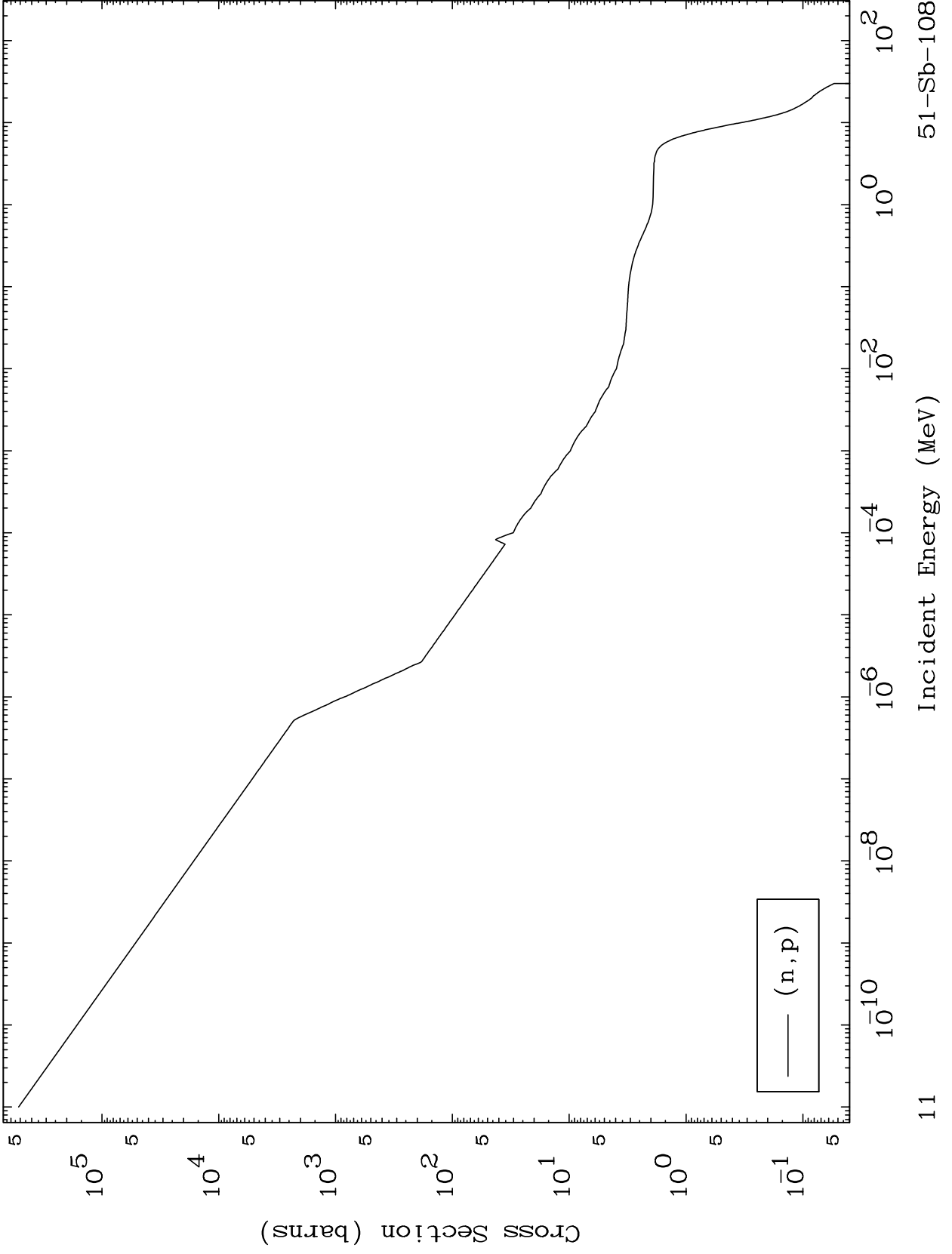
Incident Energy (MeV)

51-Sb-108

MAT 5086

(n,p) Levels  
293 Kelvin Cross Sections

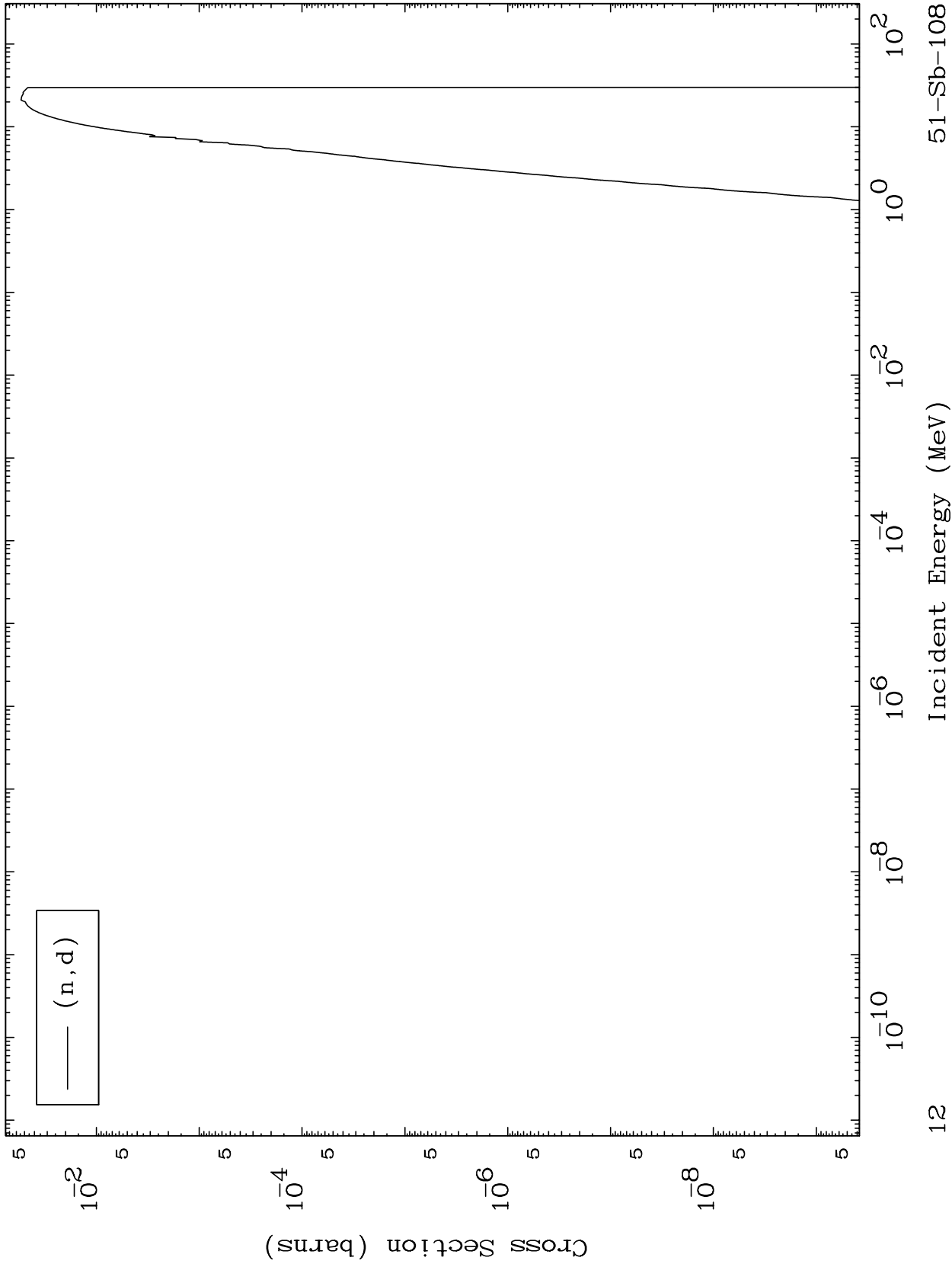
51-Sb-108



MAT 5086

(n,d) Levels  
293 Kelvin Cross Sections

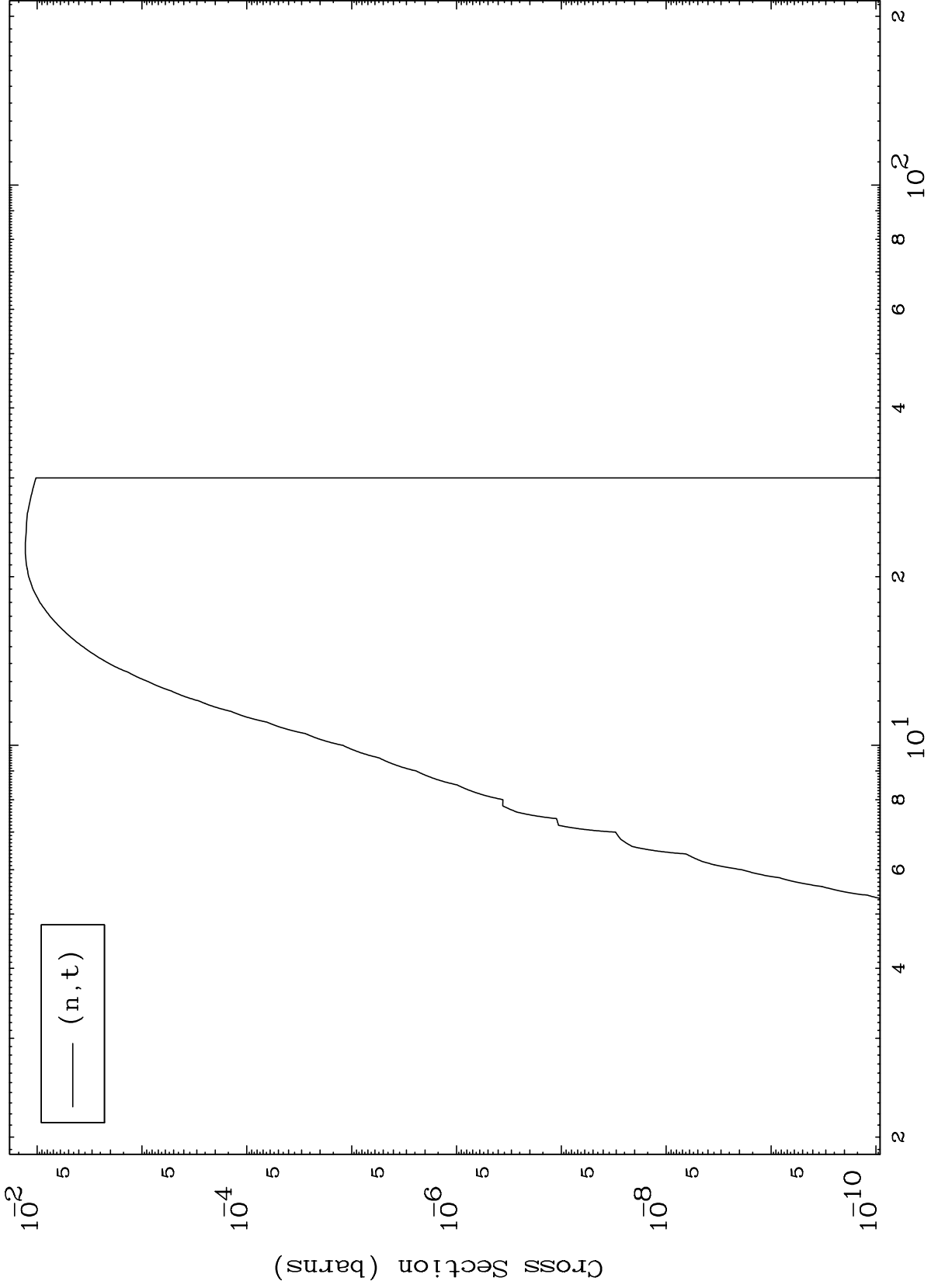
51-Sb-108



MAT 5086

(n,t) Levels  
293 Kelvin Cross Sections

51-Sb-108



13

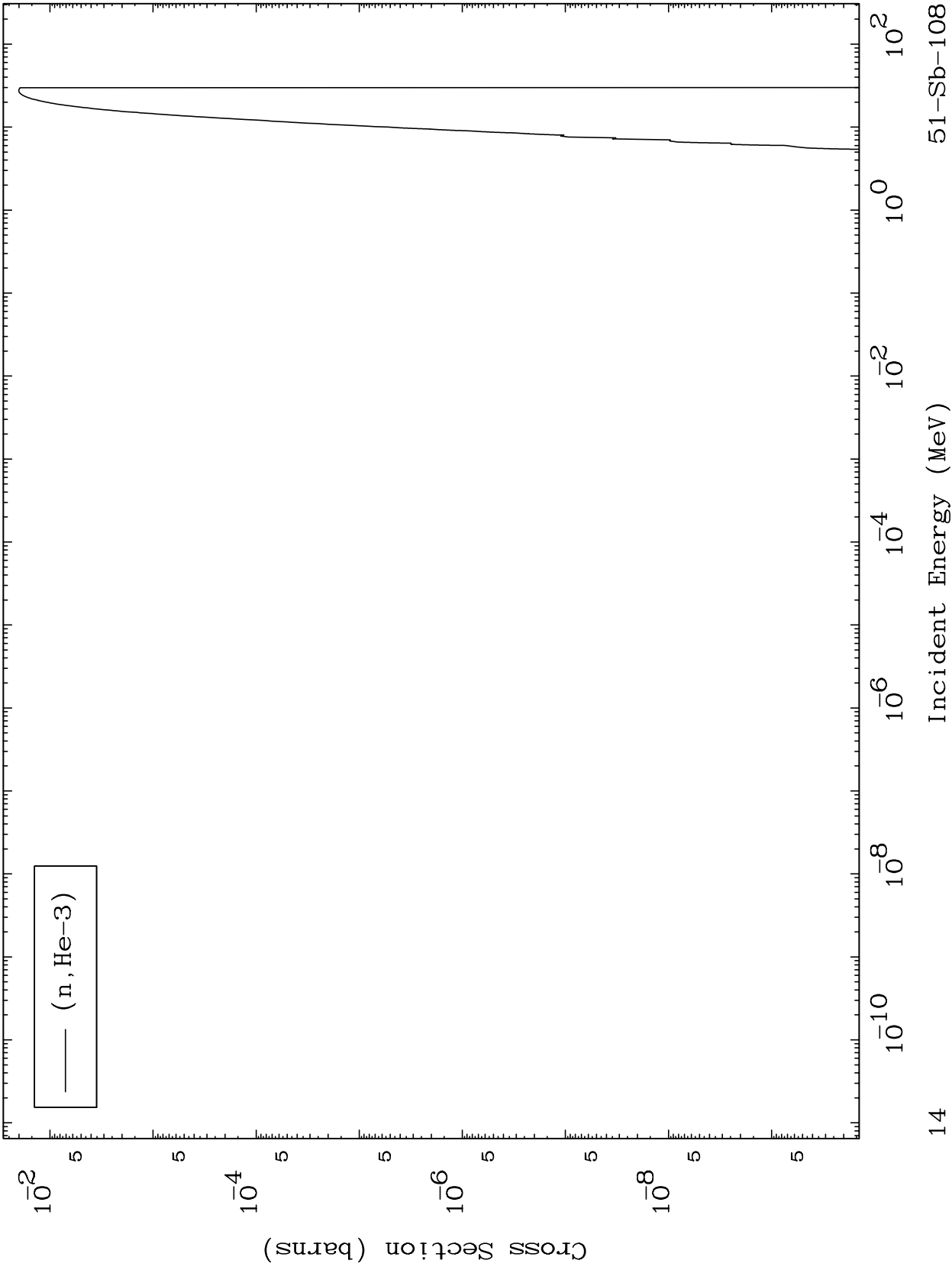
Incident Energy (MeV)

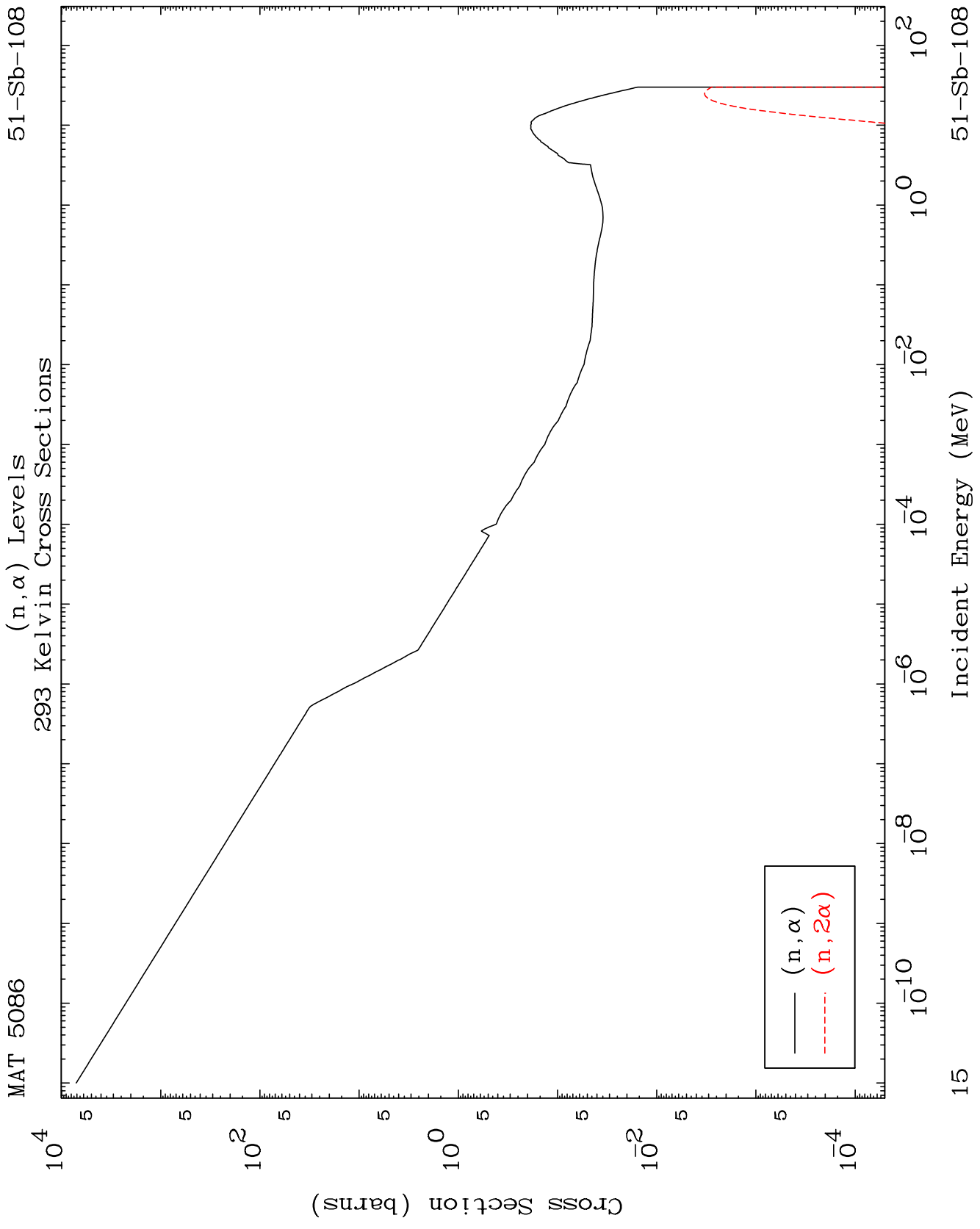
51-Sb-108

MAT 5086

(n,He3) Levels  
293 Kelvin Cross Sections

51-Sb-108



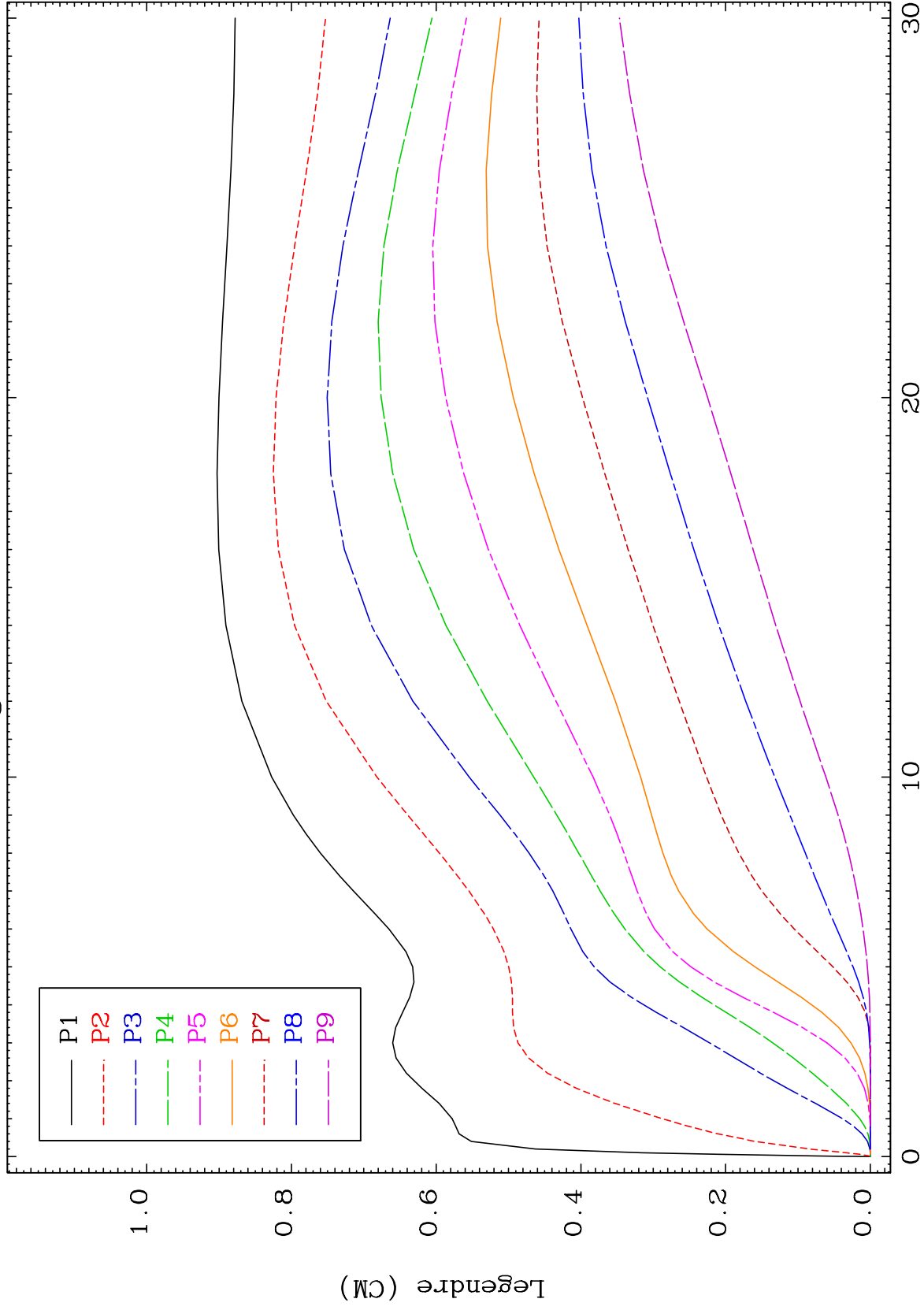




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Elastic Legendre Coefficients

51-Sb-108

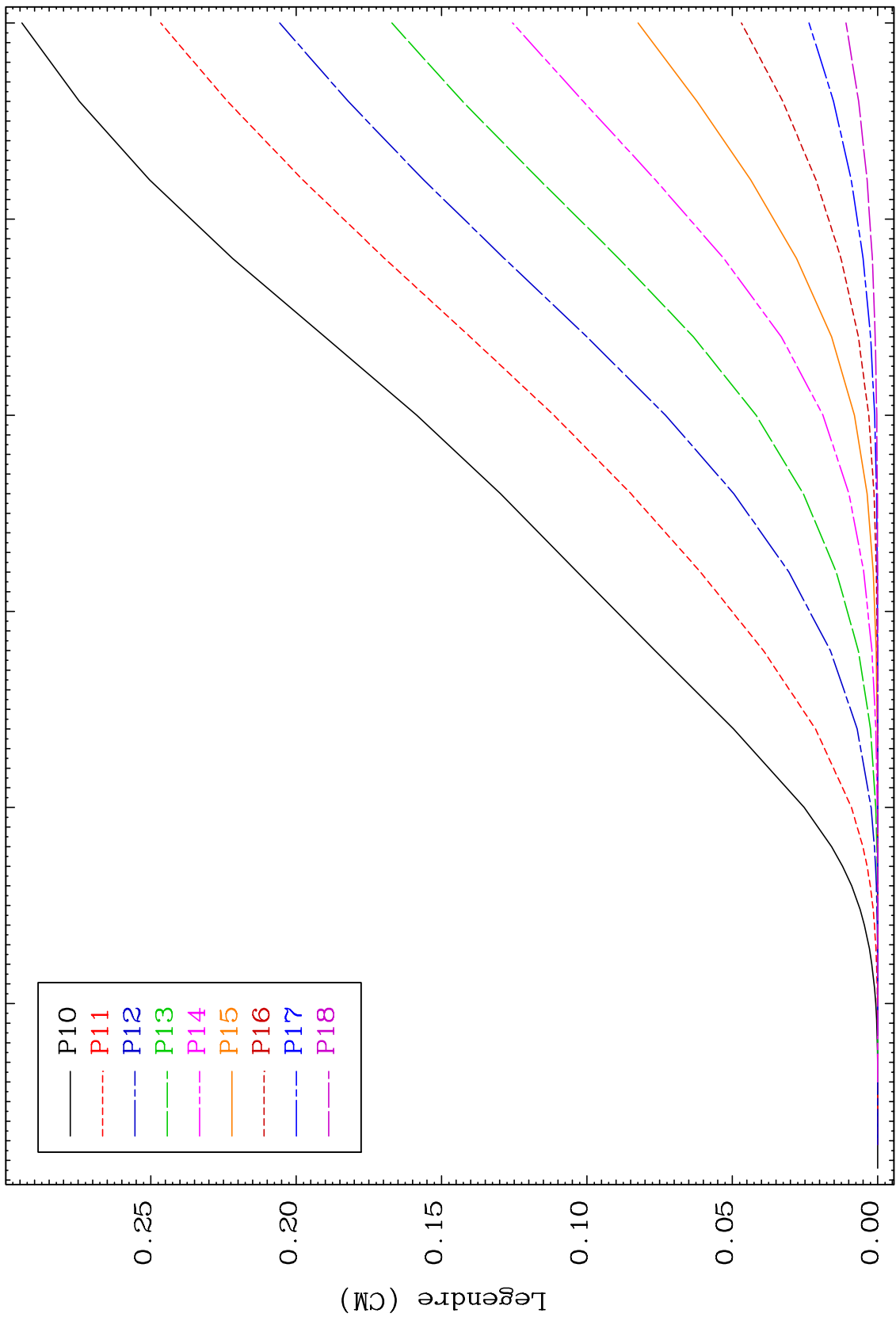


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Incident Energy (MeV)

51-Sb-108

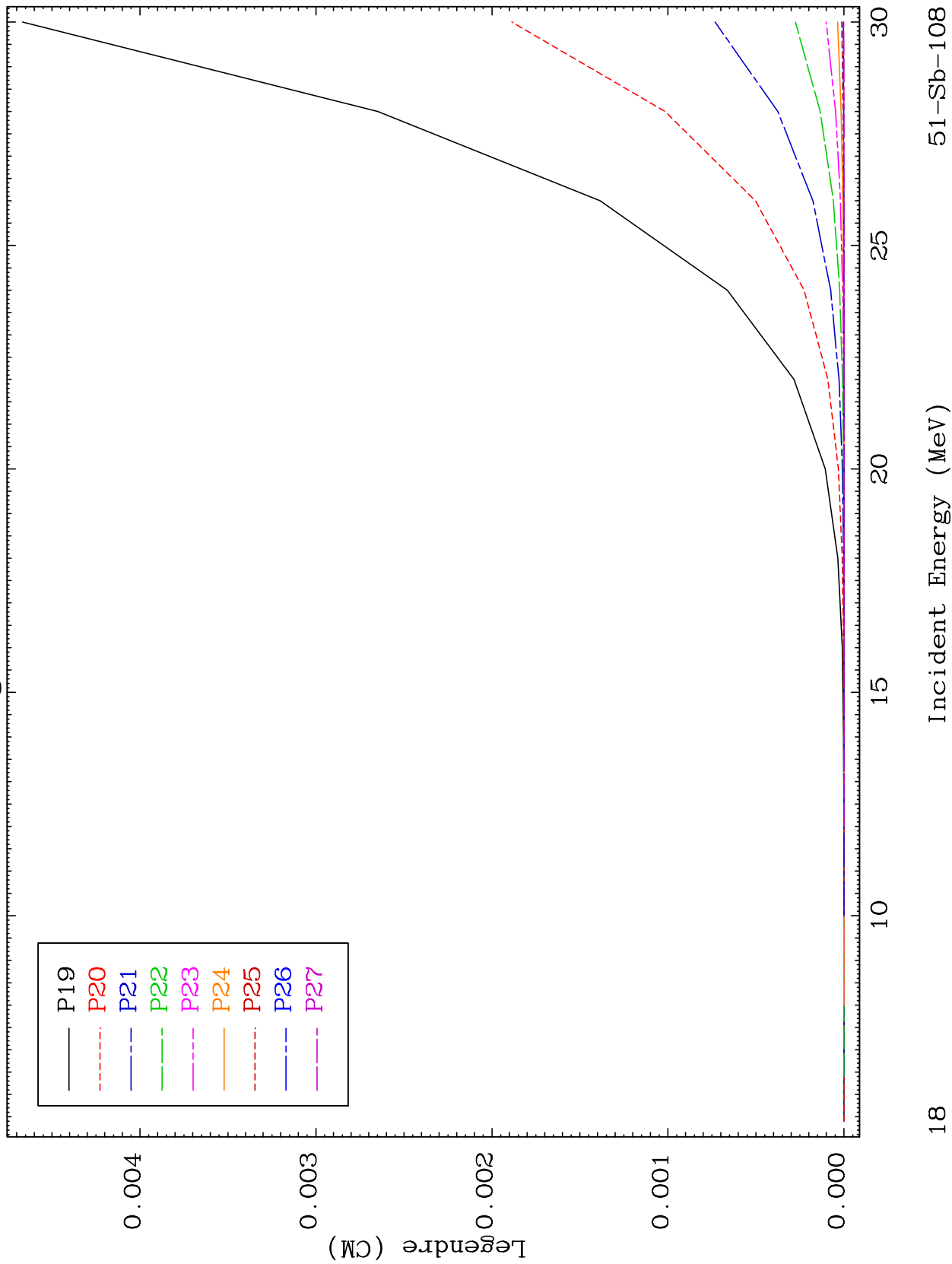
Elastic Legendre Coefficients



MAT 5086

Elastic Legendre Coefficients

51-Sb-108



18

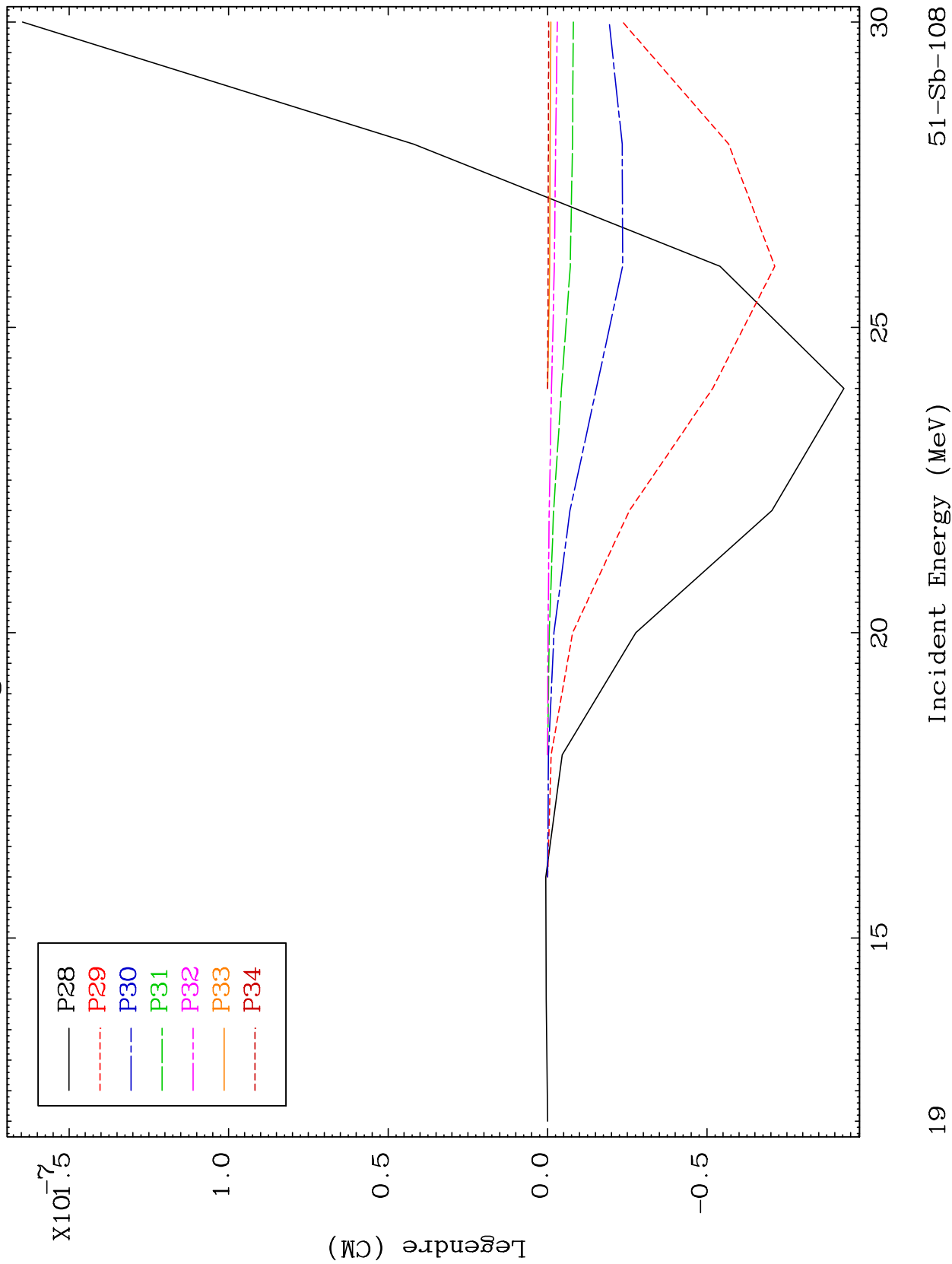
Incident Energy (MeV)

51-Sb-108

MAT 5086

Elastic Legendre Coefficients

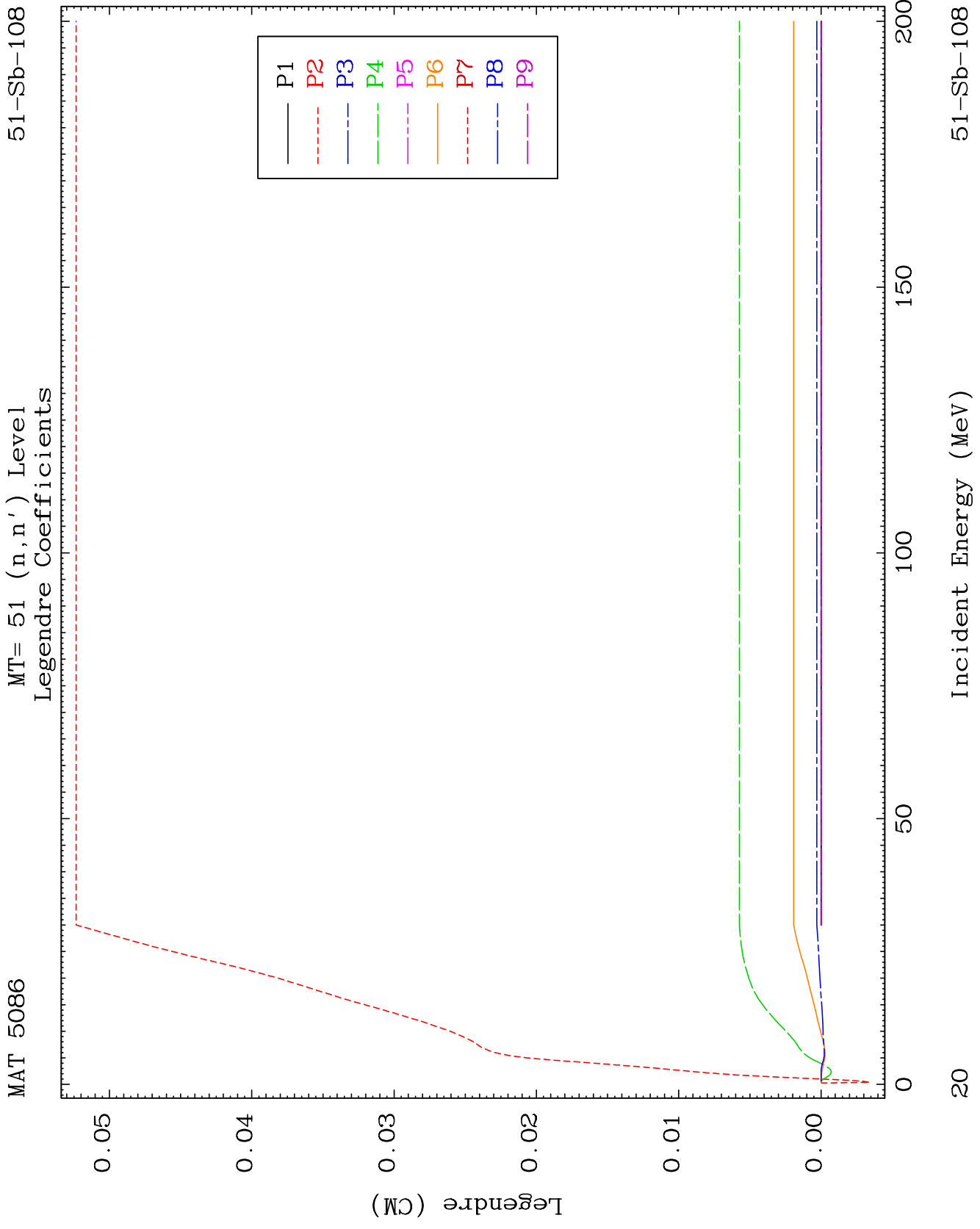
51-Sb-108



19

Incident Energy (MeV)

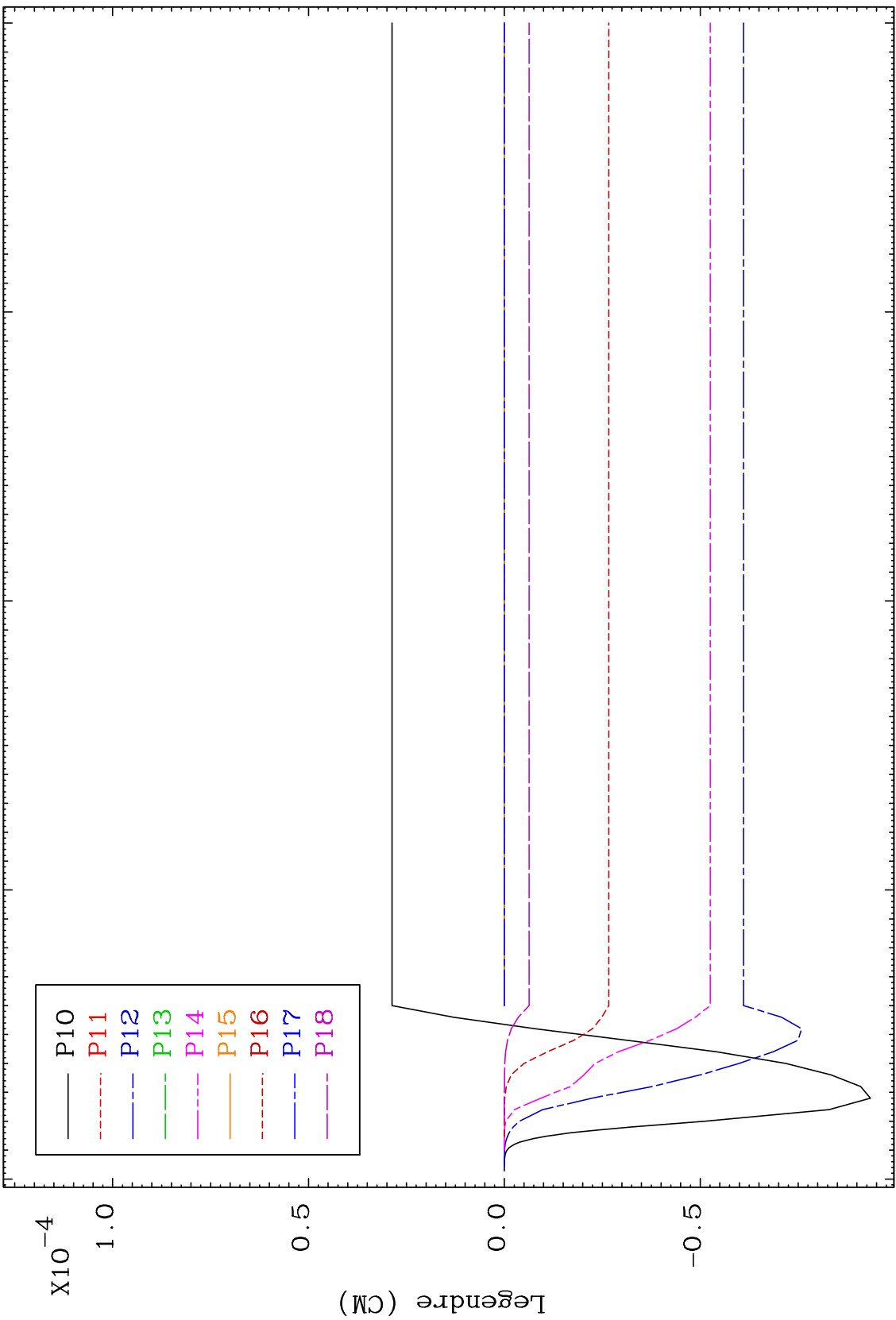
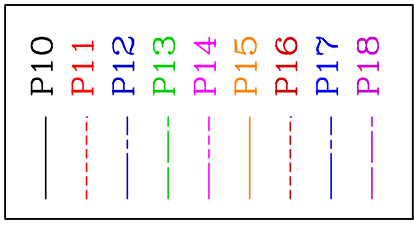
51-Sb-108



MAT 5086

MT= 51 (n,n') Level  
Legendre Coefficients

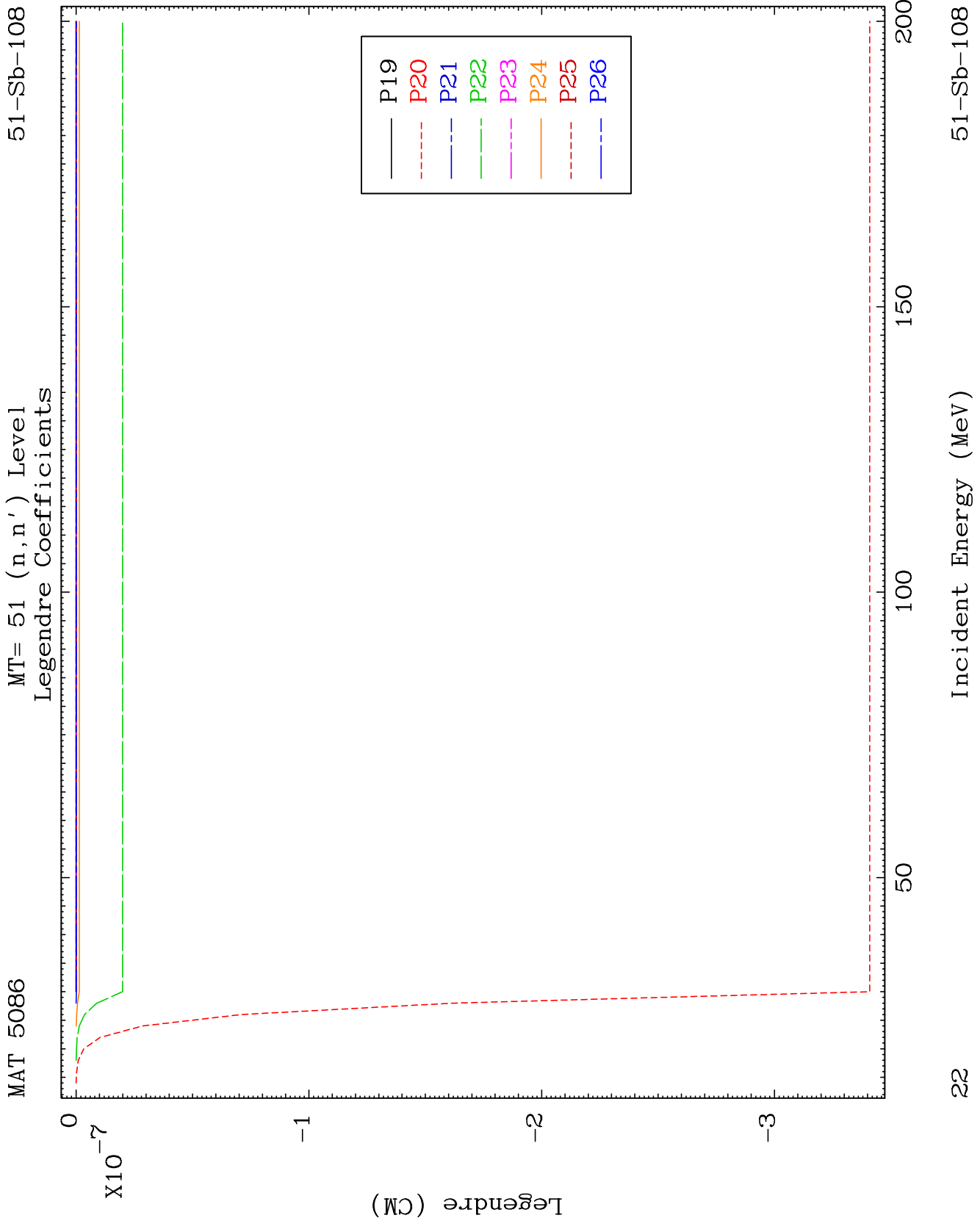
51-Sb-108



21

Incident Energy (MeV)

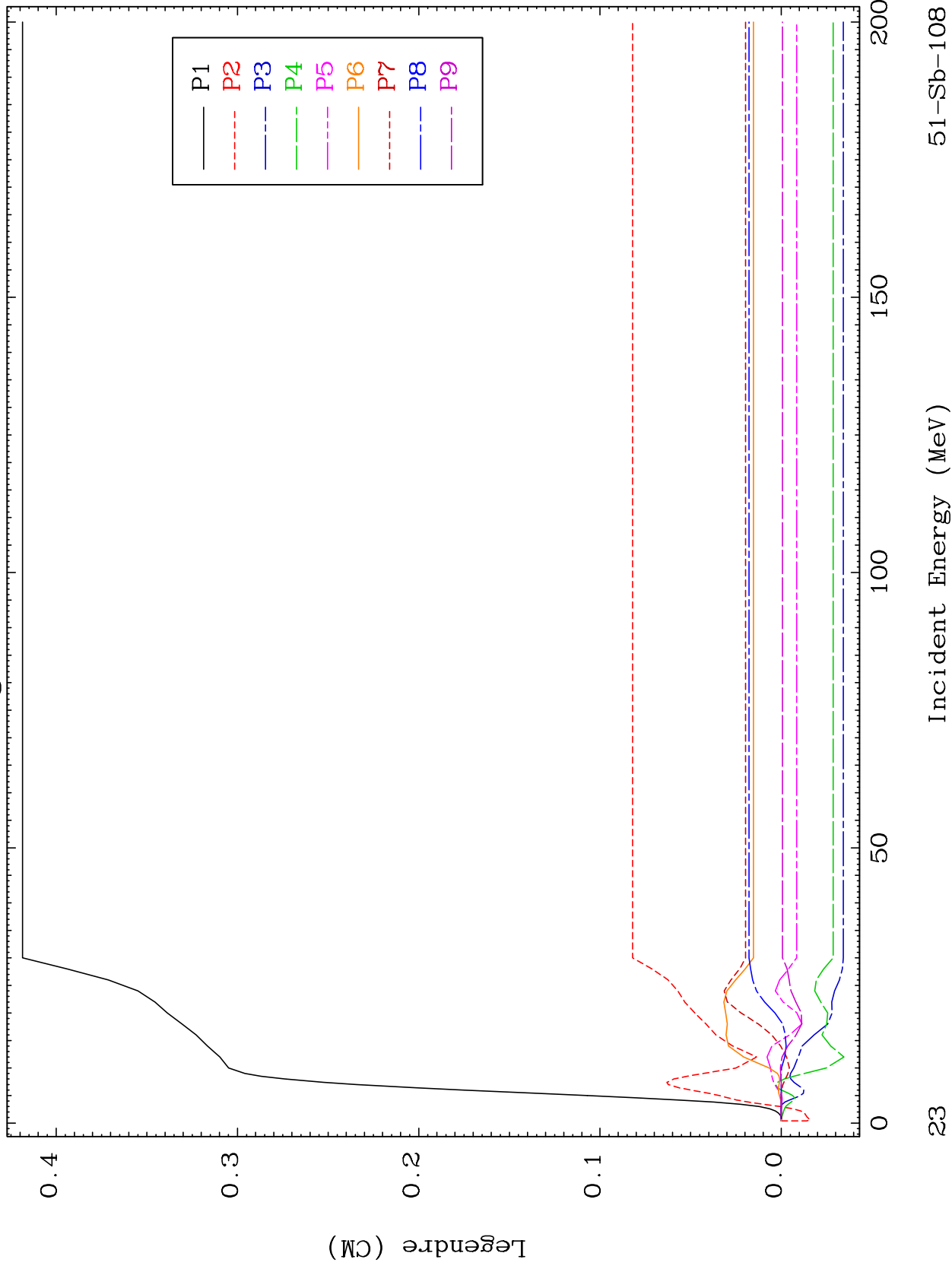
51-Sb-108



MAT 5086

MT= 52 (n,n') Level  
Legendre Coefficients

51-Sb-108



23

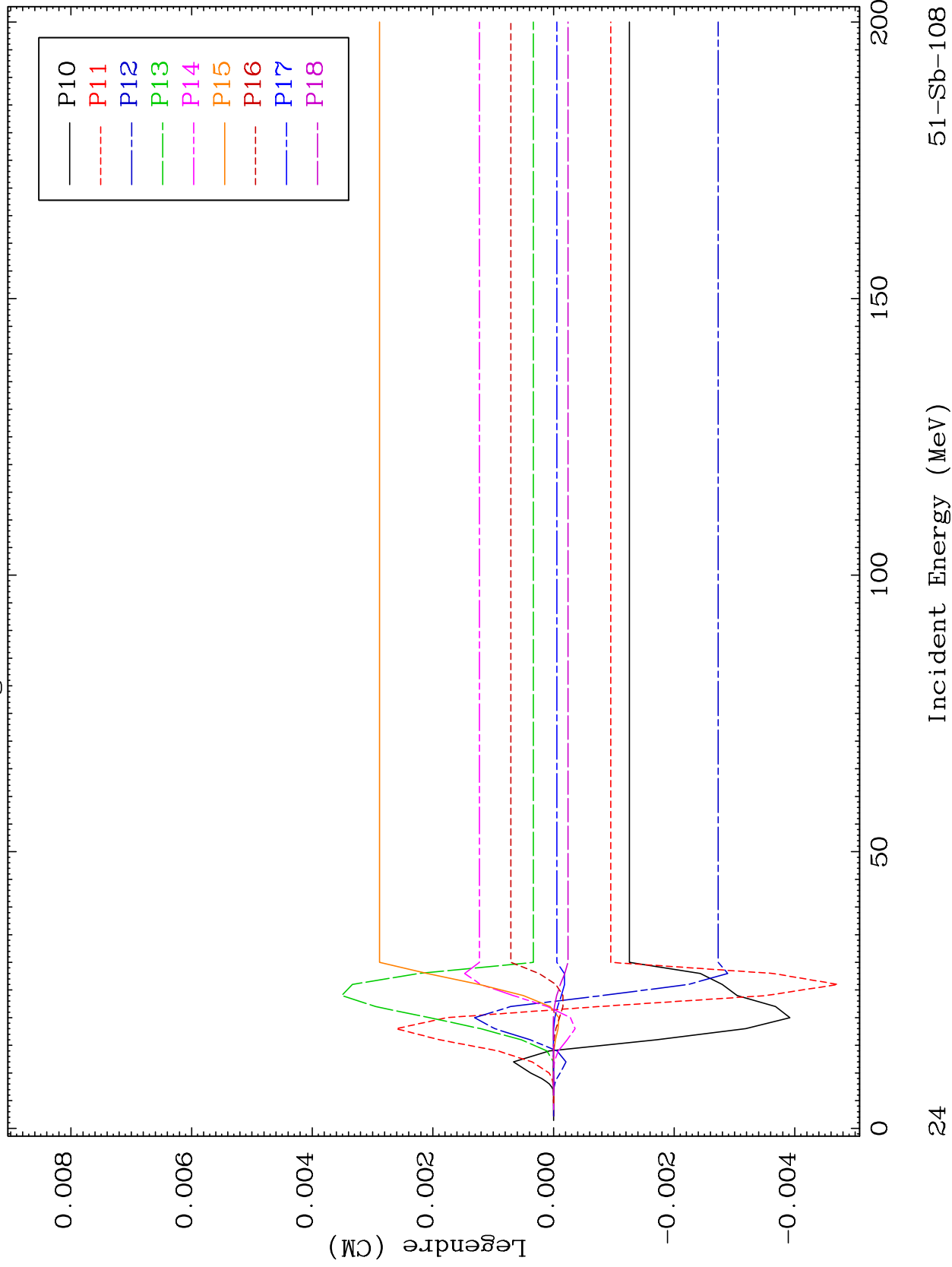
51-Sb-108



MAT 5086

MT= 52 (n,n') Level  
Legendre Coefficients

51-Sb-108



51-Sb-108

Incident Energy (MeV)

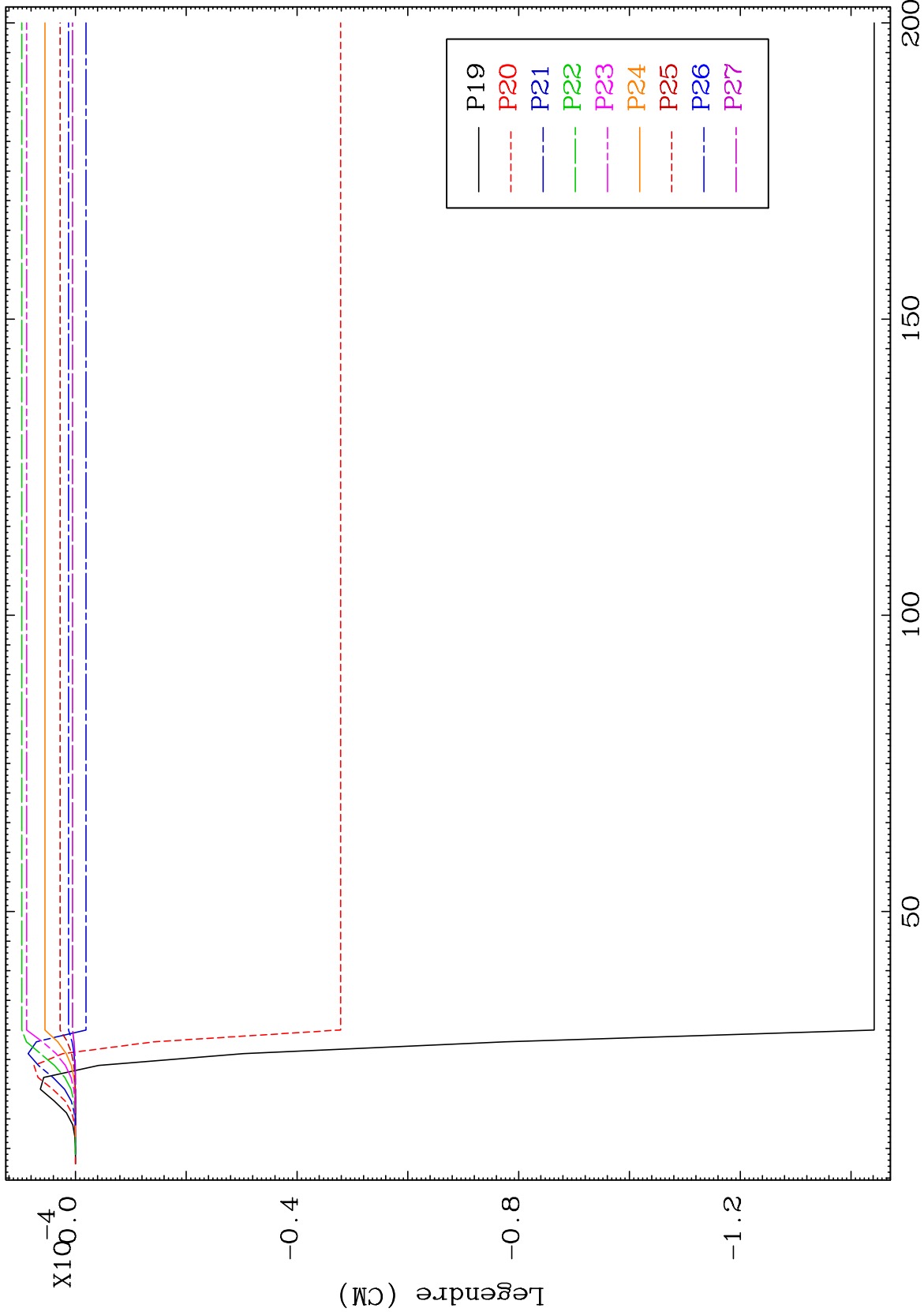
24

MAT 5086

MT= 52 (n,n') Level

51-Sb-108

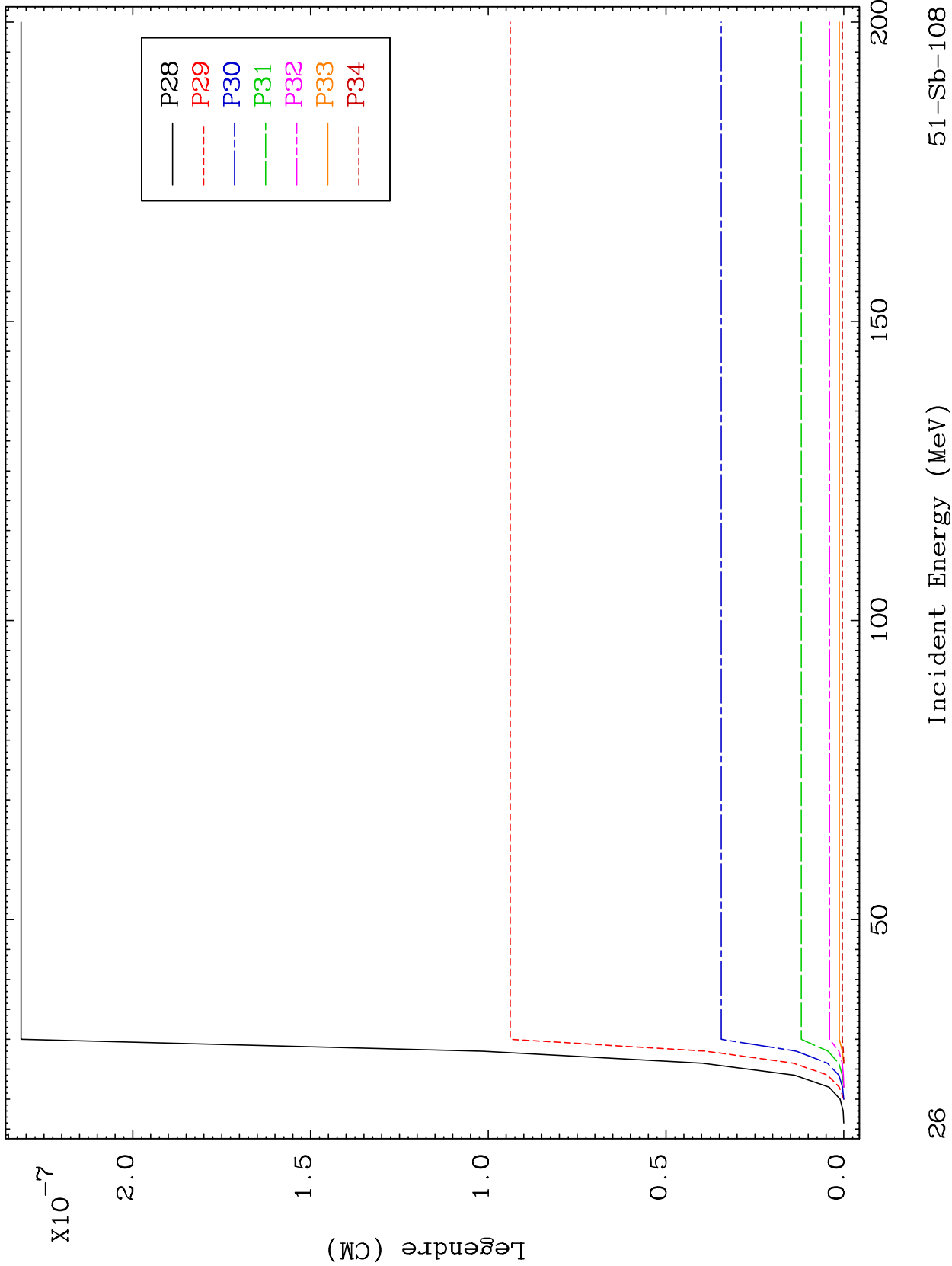
Legendre Coefficients



25

Incident Energy (MeV)

51-Sb-108

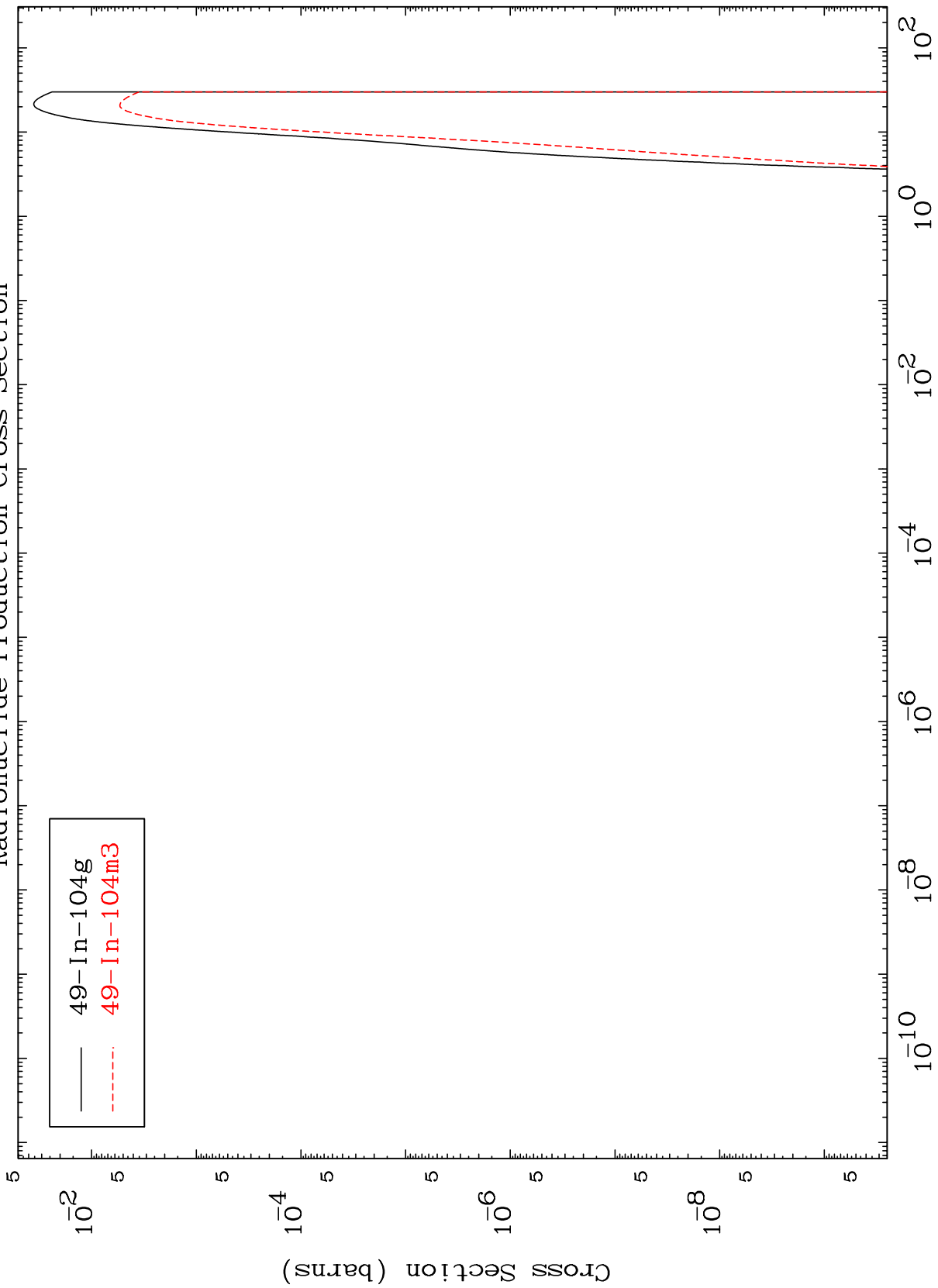


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(n,n')  $\alpha$

51-Sb-108

Radionuclide Production Cross Section

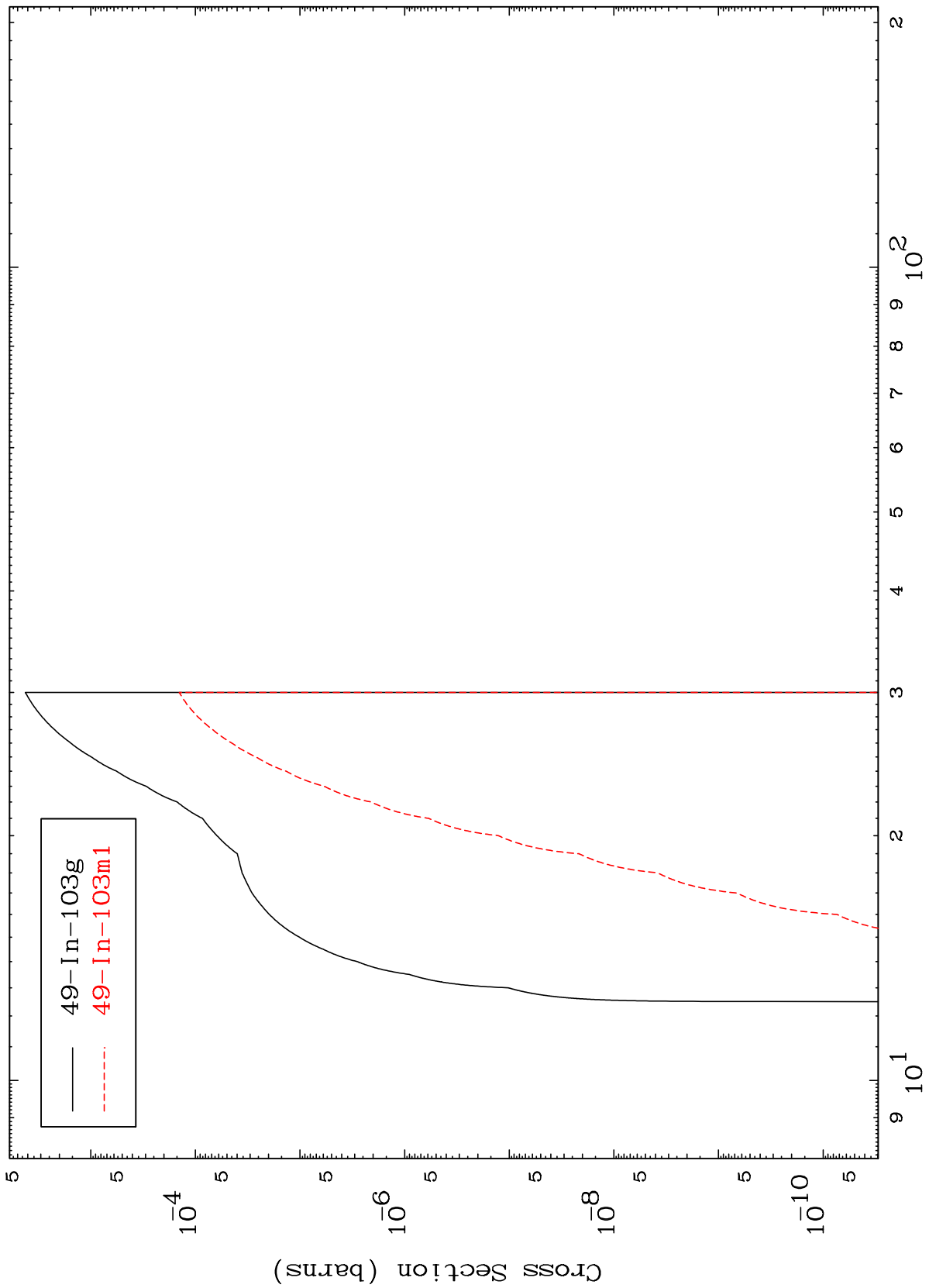


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(n,2n)  $\alpha$

51-Sb-108

Radionuclide Production Cross Section



49-In-103g  
49-In-103m1

28

Incident Energy (MeV)

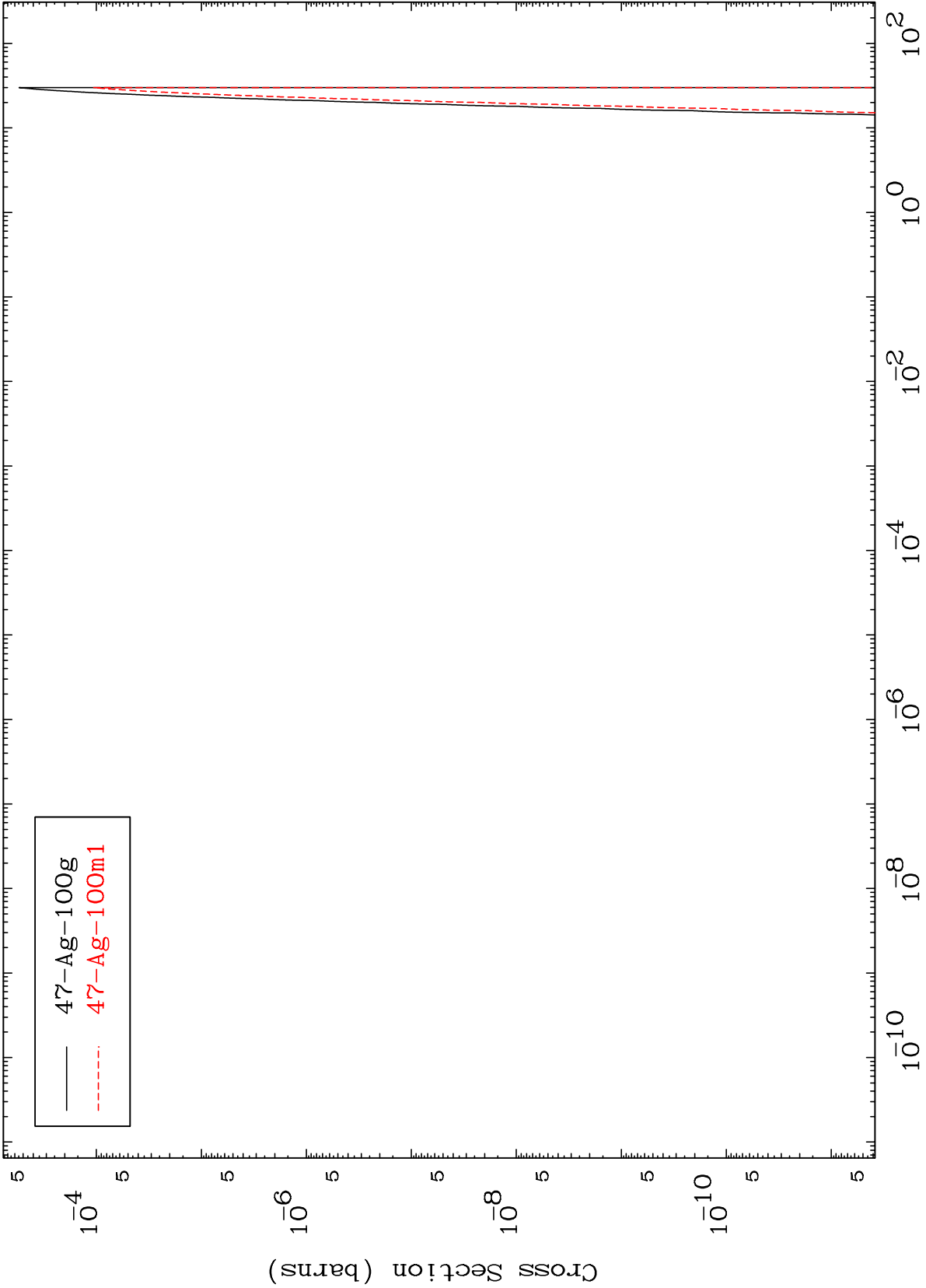
51-Sb-108

MAT 5086

(n,n') 2α

51-Sb-108

Radionuclide Production Cross Section

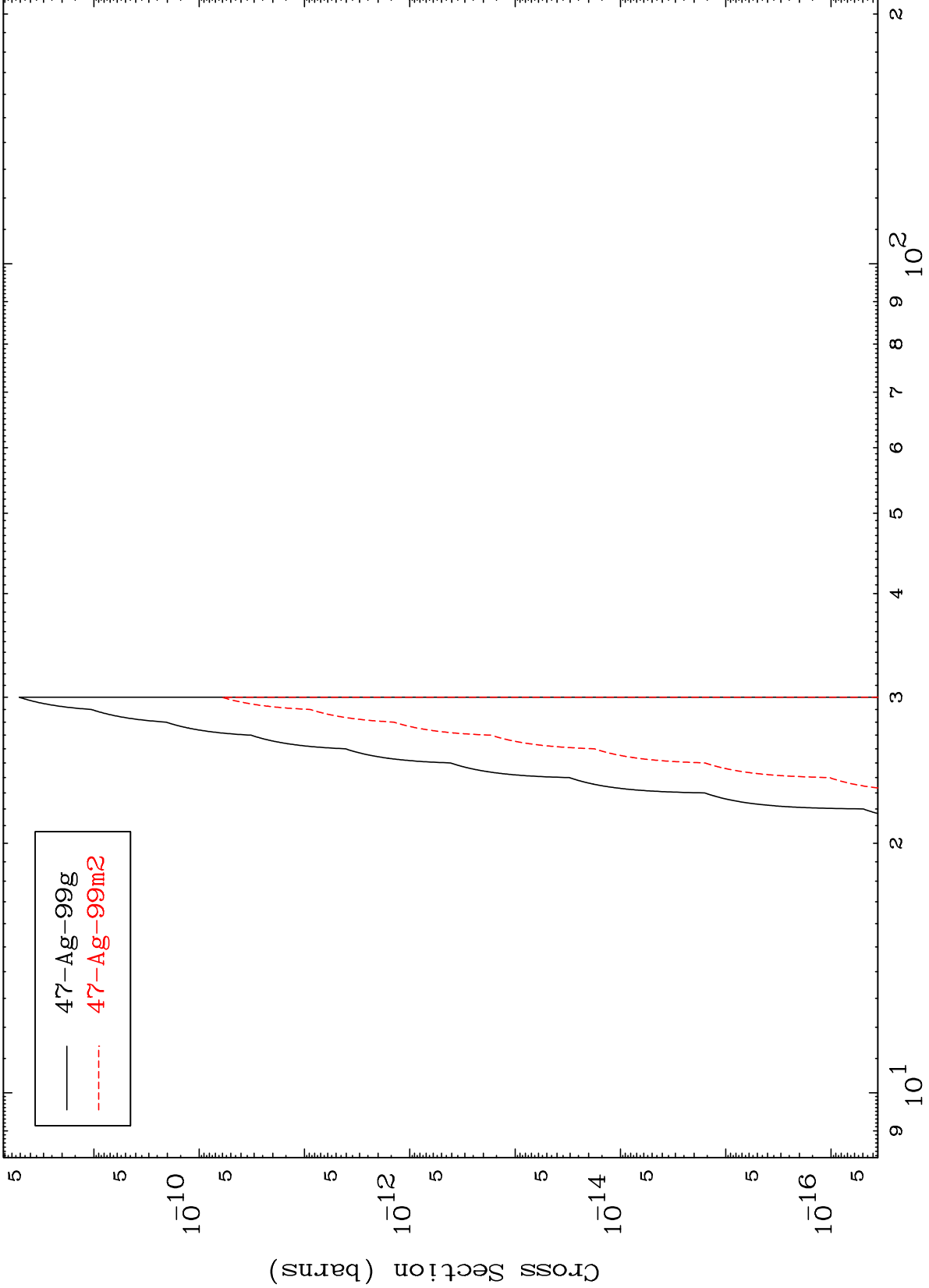


MAT 5086

(n,2n) 2α

51-Sb-108

Radionuclide Production Cross Section



— 47-Ag-99g  
- - - 47-Ag-99m2

30

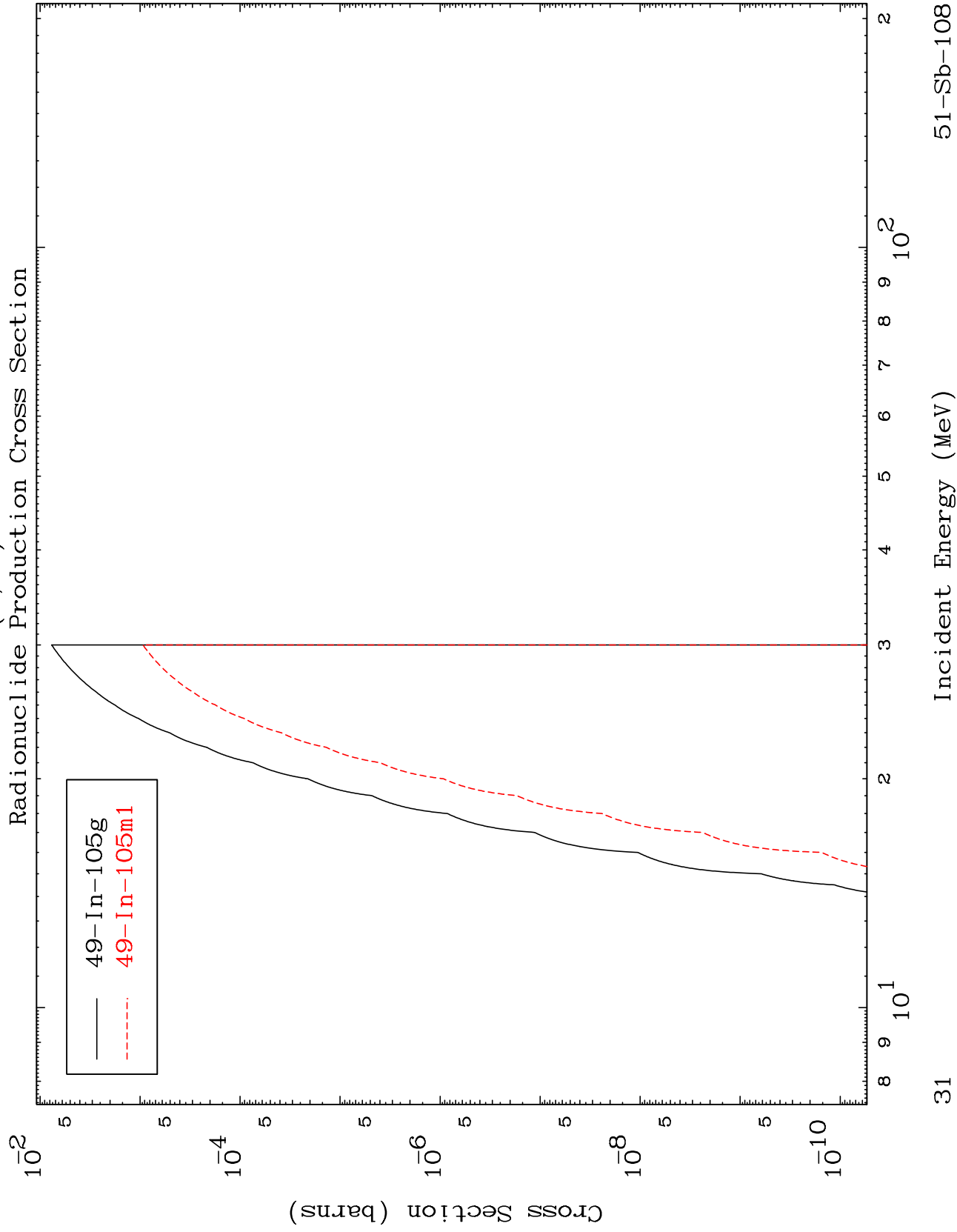
Incident Energy (MeV)

51-Sb-108

MAT 5086

(n,n') He-3

51-Sb-108



31

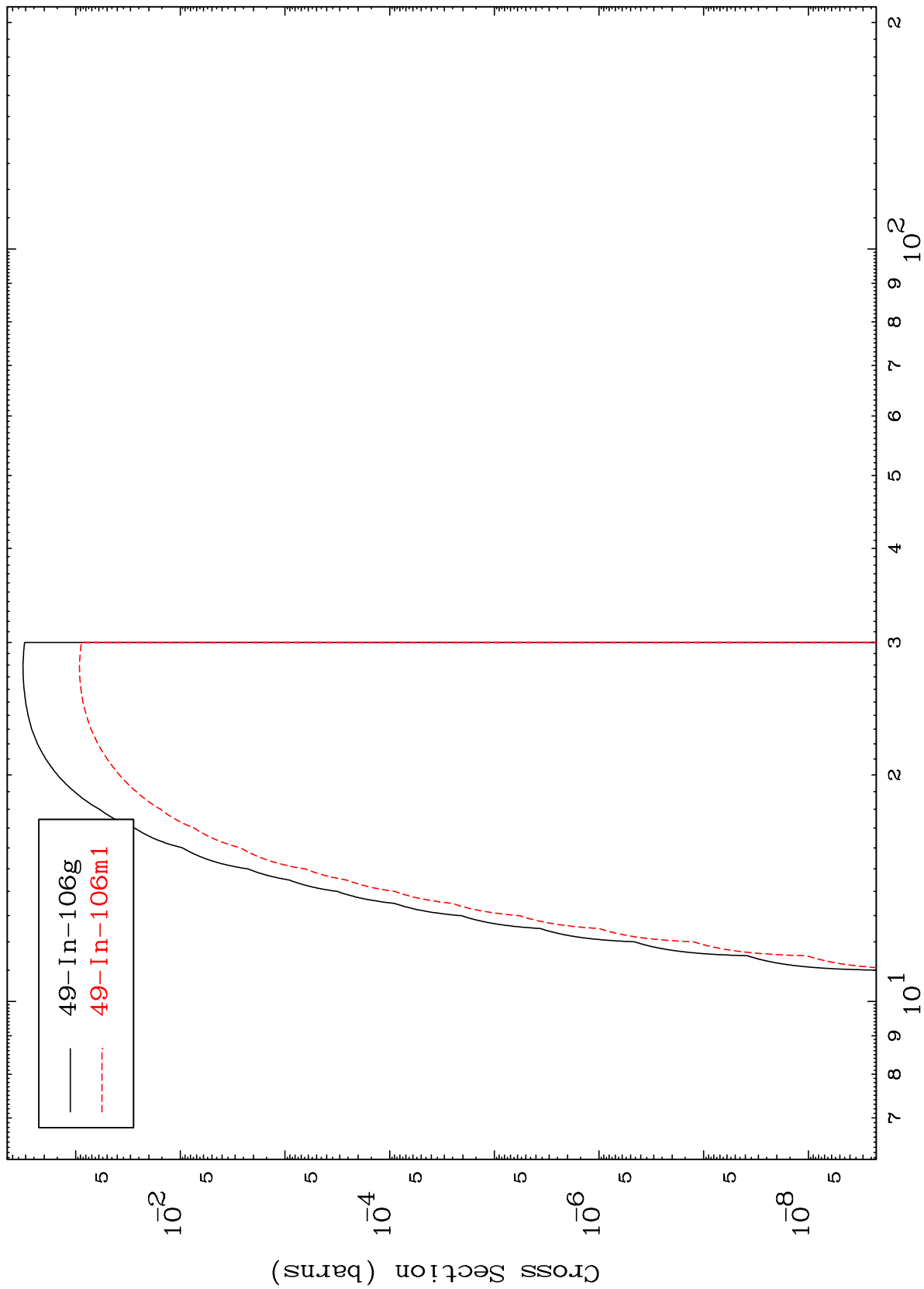


MAT 5086

(n,2n) p

51-Sb-108

Radionuclide Production Cross Section



32

Incident Energy (MeV)

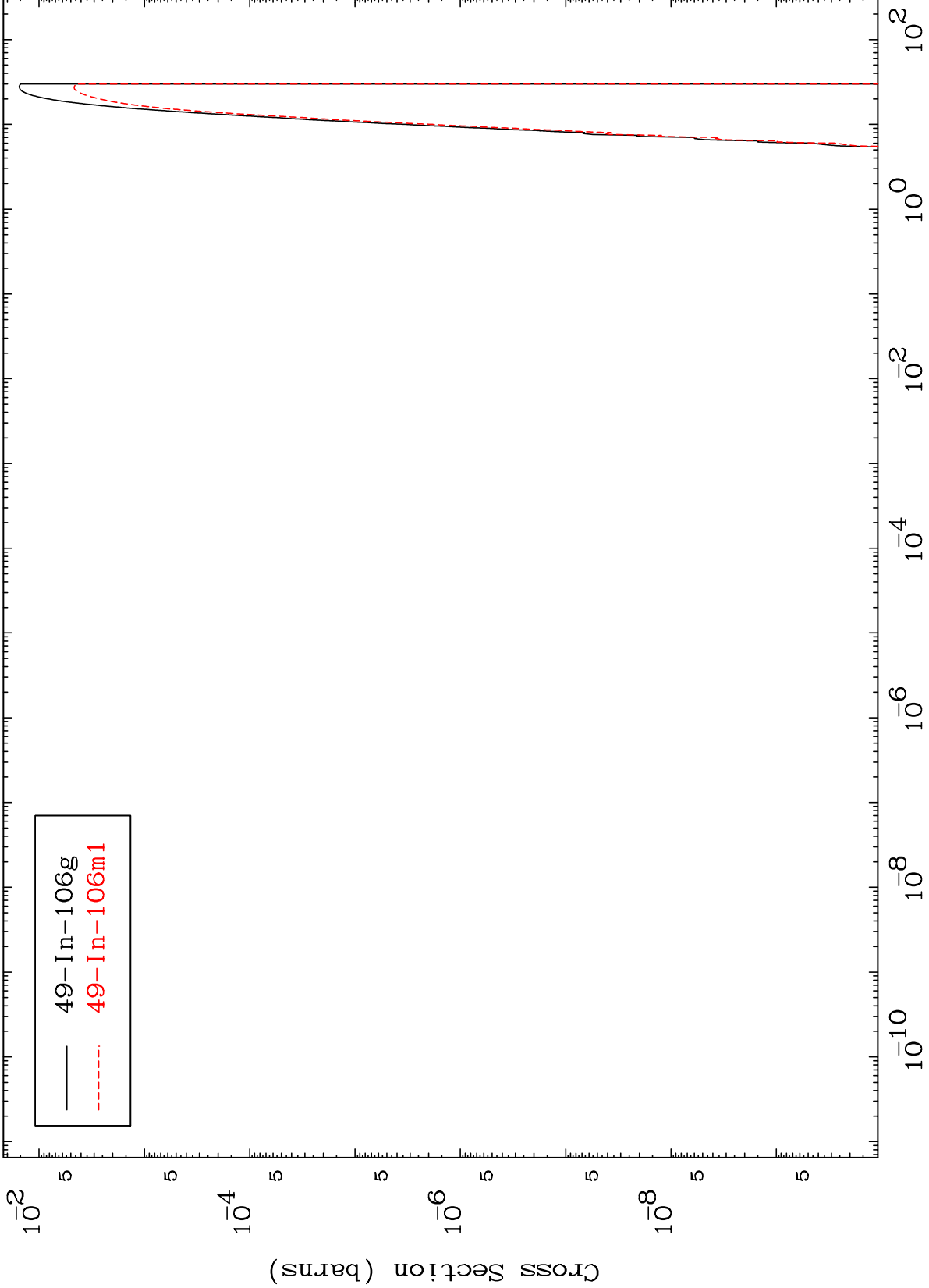
51-Sb-108

MAT 5086

(n,He-3)

51-Sb-108

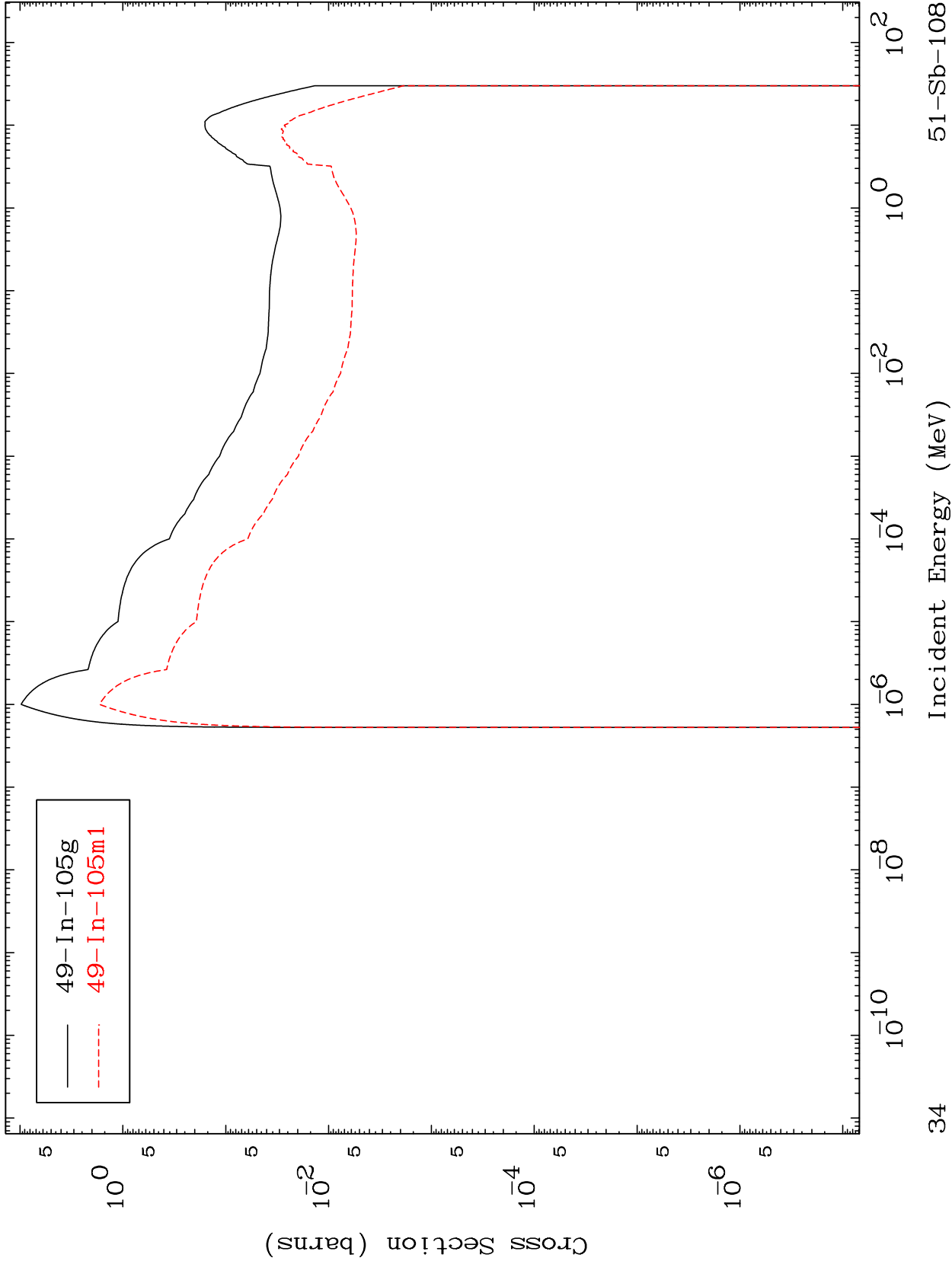
Radionuclide Production Cross Section



MAT 5086

51-Sb-108

Radionuclide Production Cross Section

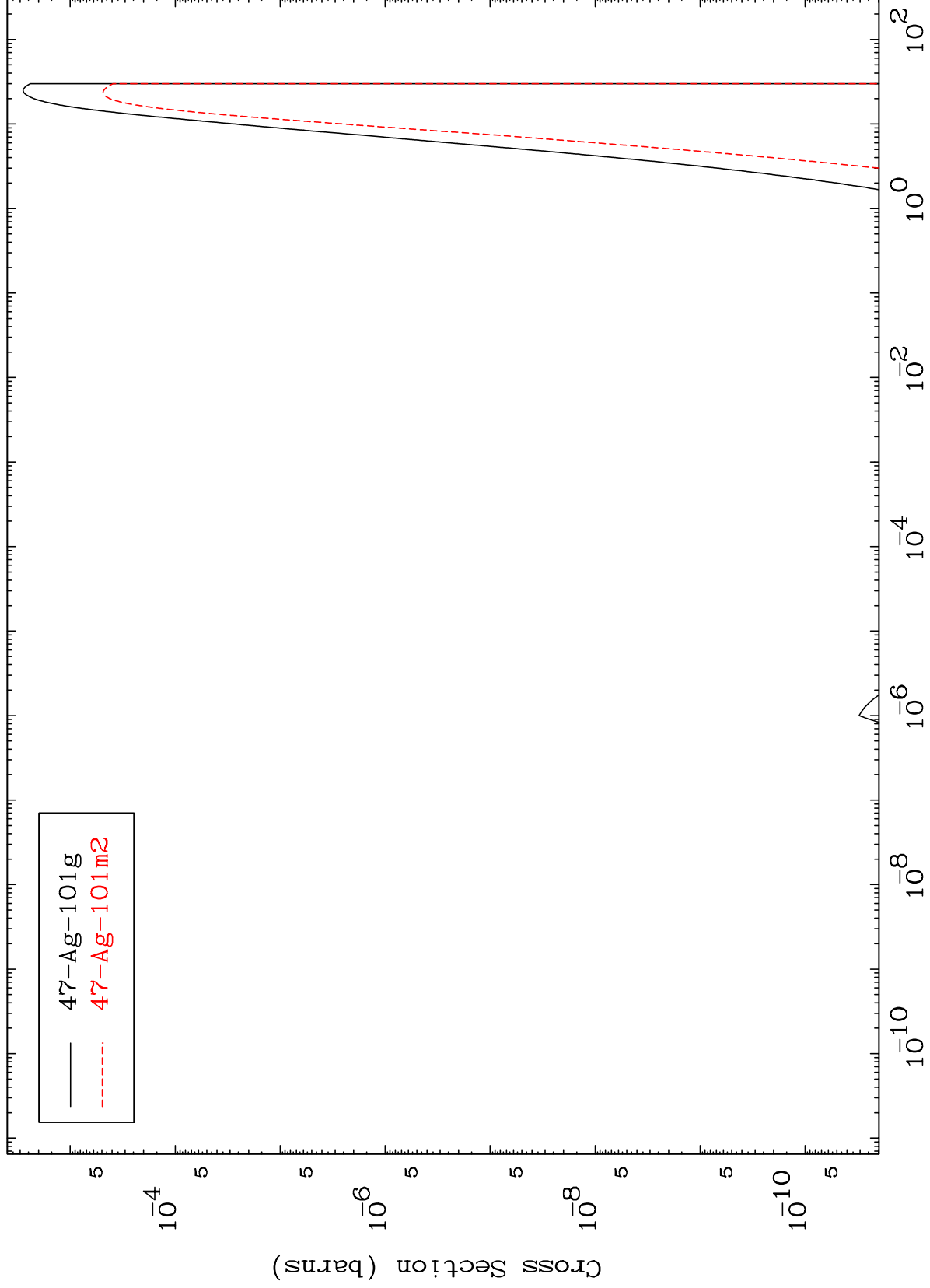


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(n,2α)

51-Sb-108

Radionuclide Production Cross Section



35

Incident Energy (MeV)

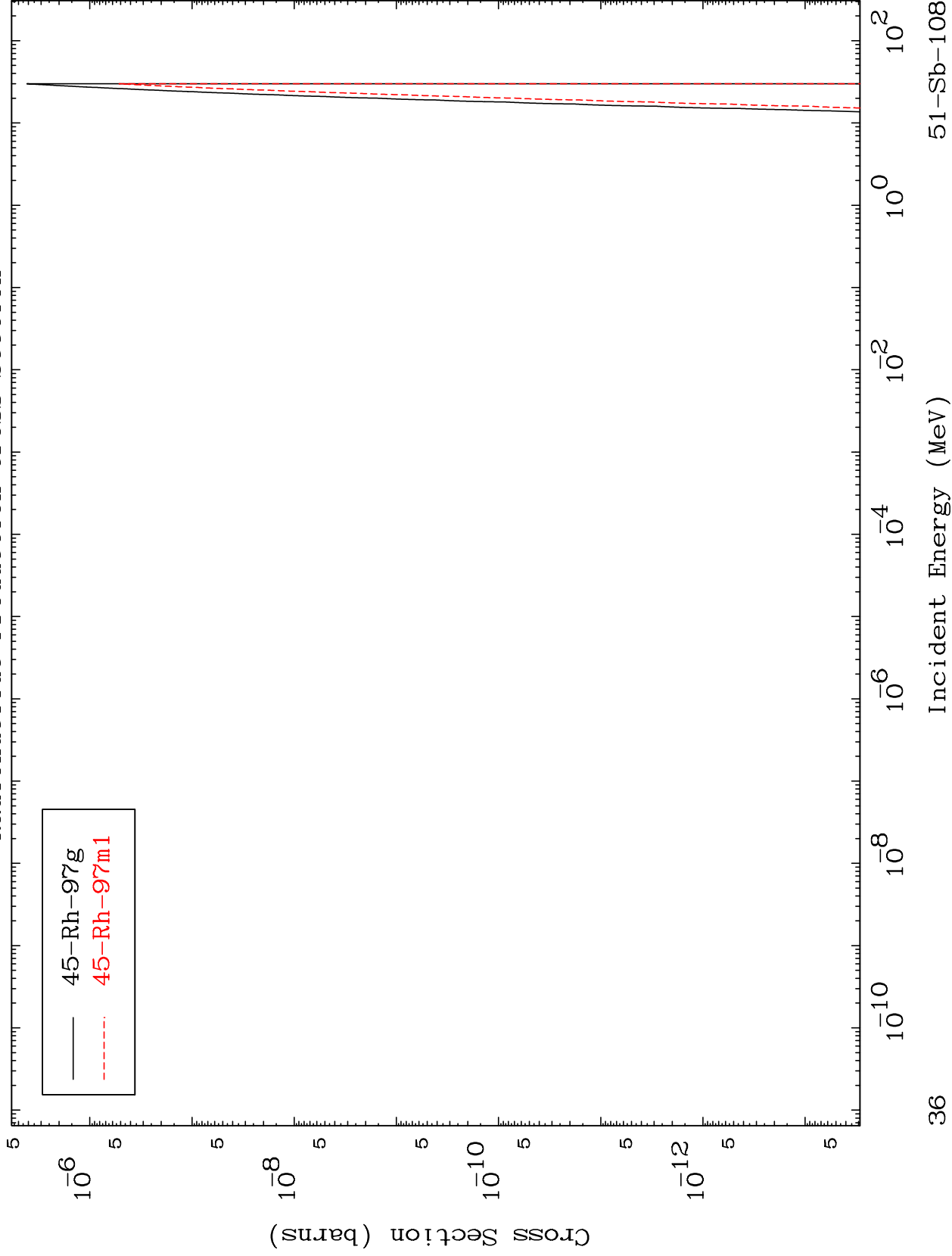
51-Sb-108

MAT 5086

(n,3 $\alpha$ )

51-Sb-108

Radionuclide Production Cross Section

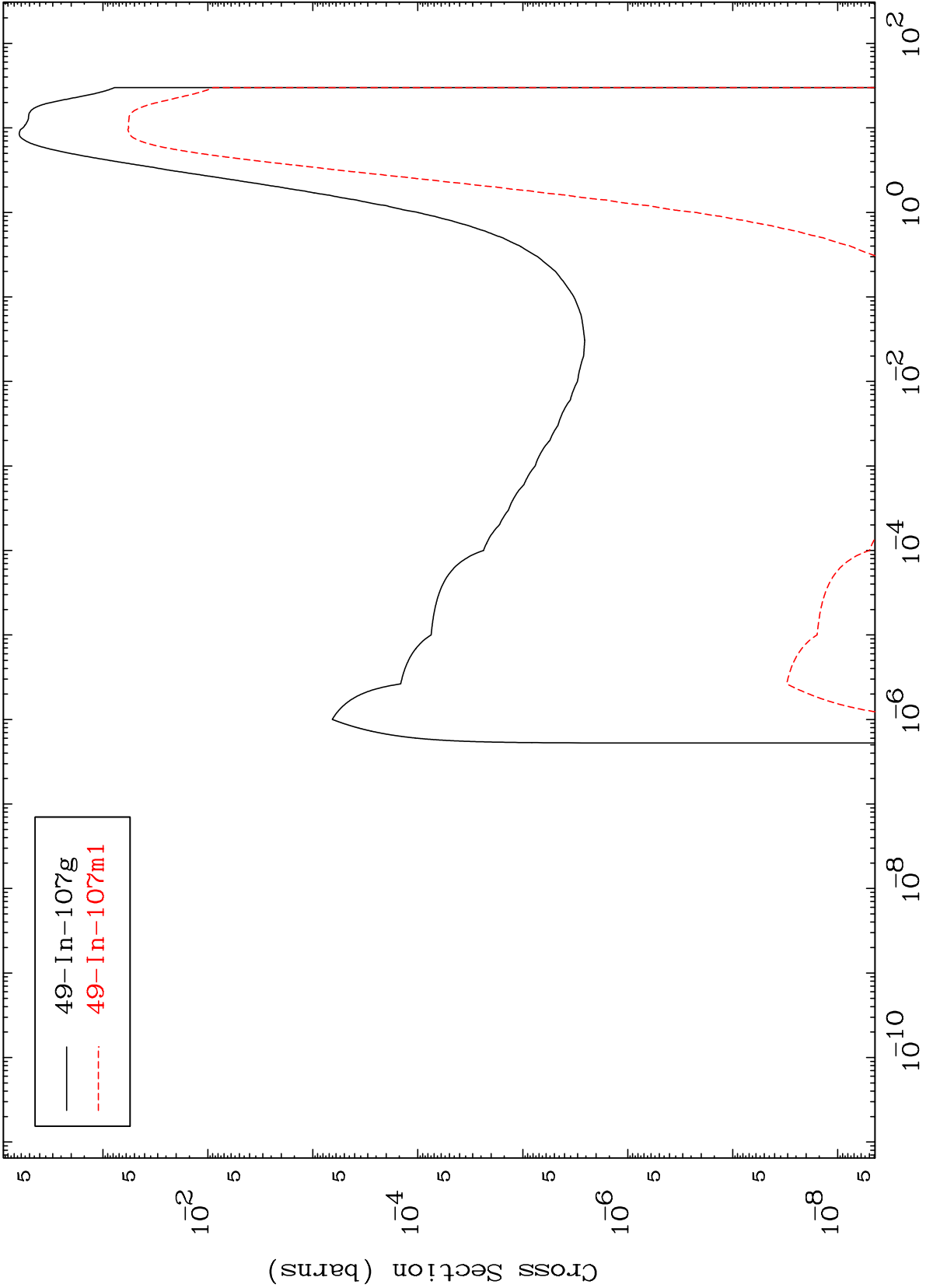


MAT 5086

(n,2p)

51-Sb-108

Radionuclide Production Cross Section

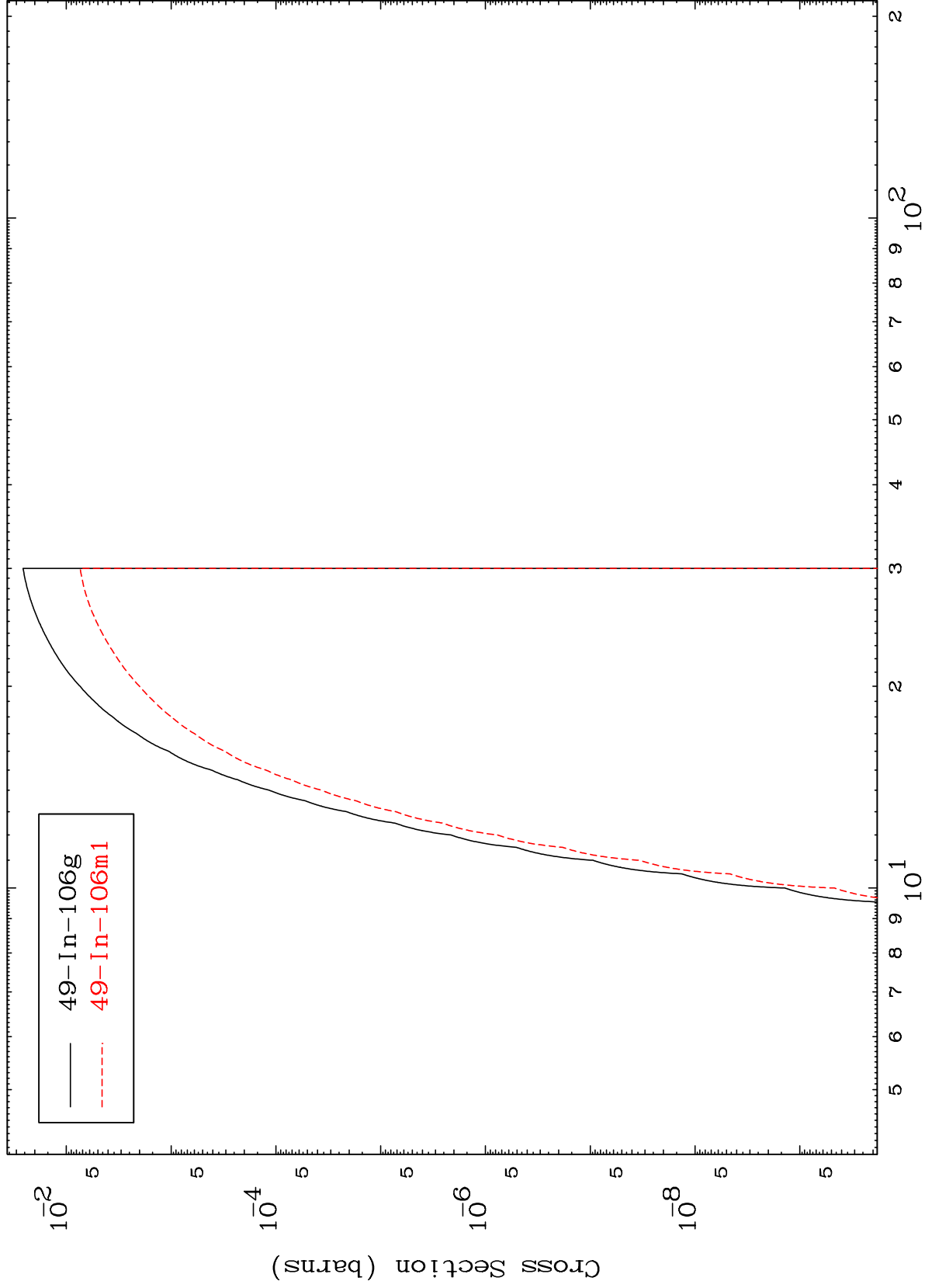


MAT 5086

(n,p) d

51-Sb-108

Radionuclide Production Cross Section



38

Incident Energy (MeV)

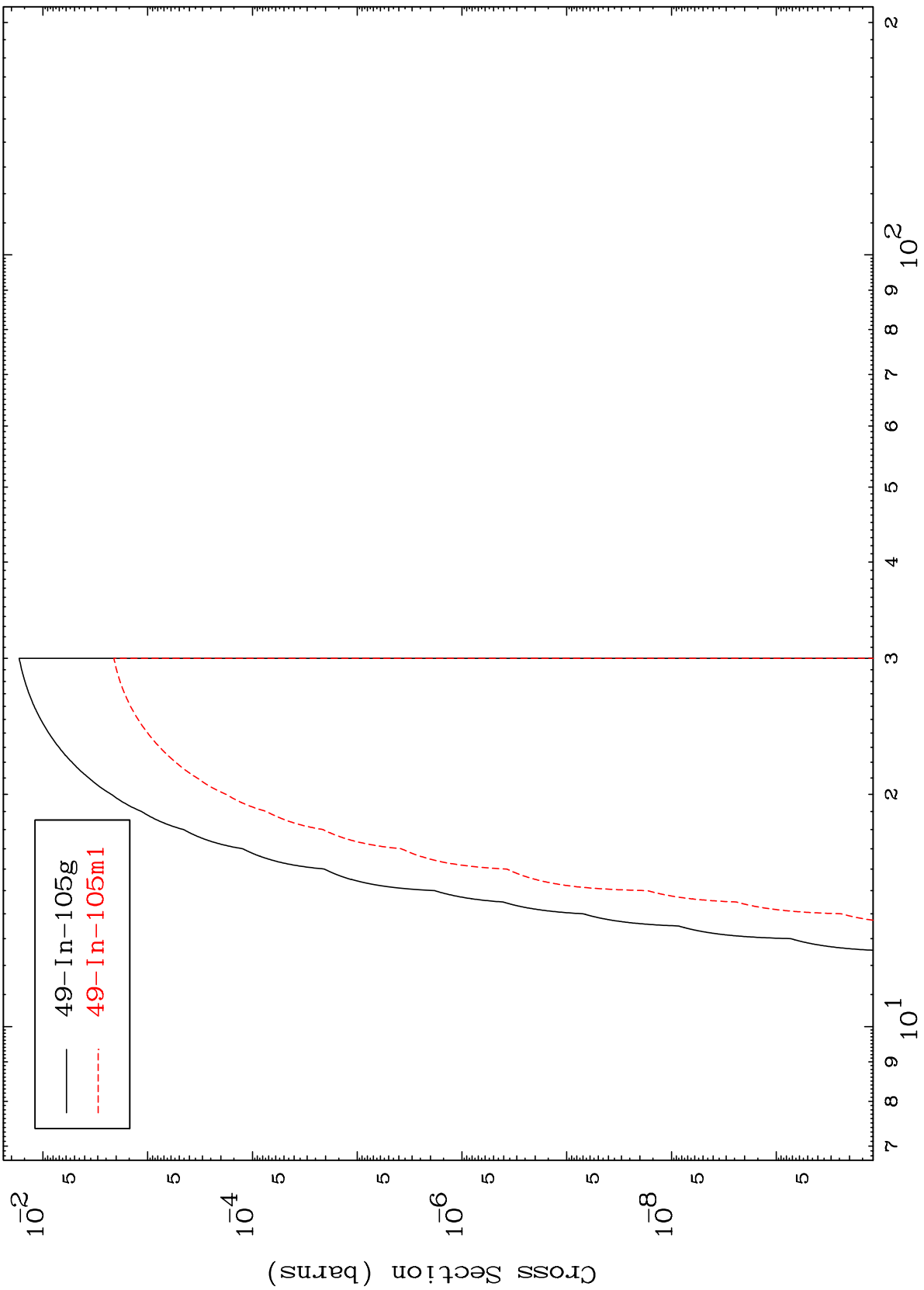
51-Sb-108

MAT 5086

(n,p) t

51-Sb-108

Radionuclide Production Cross Section



— 49-In-105g  
- - - 49-In-105m1

39

Incident Energy (MeV)

51-Sb-108