

Program EVALPLOT
(Version 2021-1)

by

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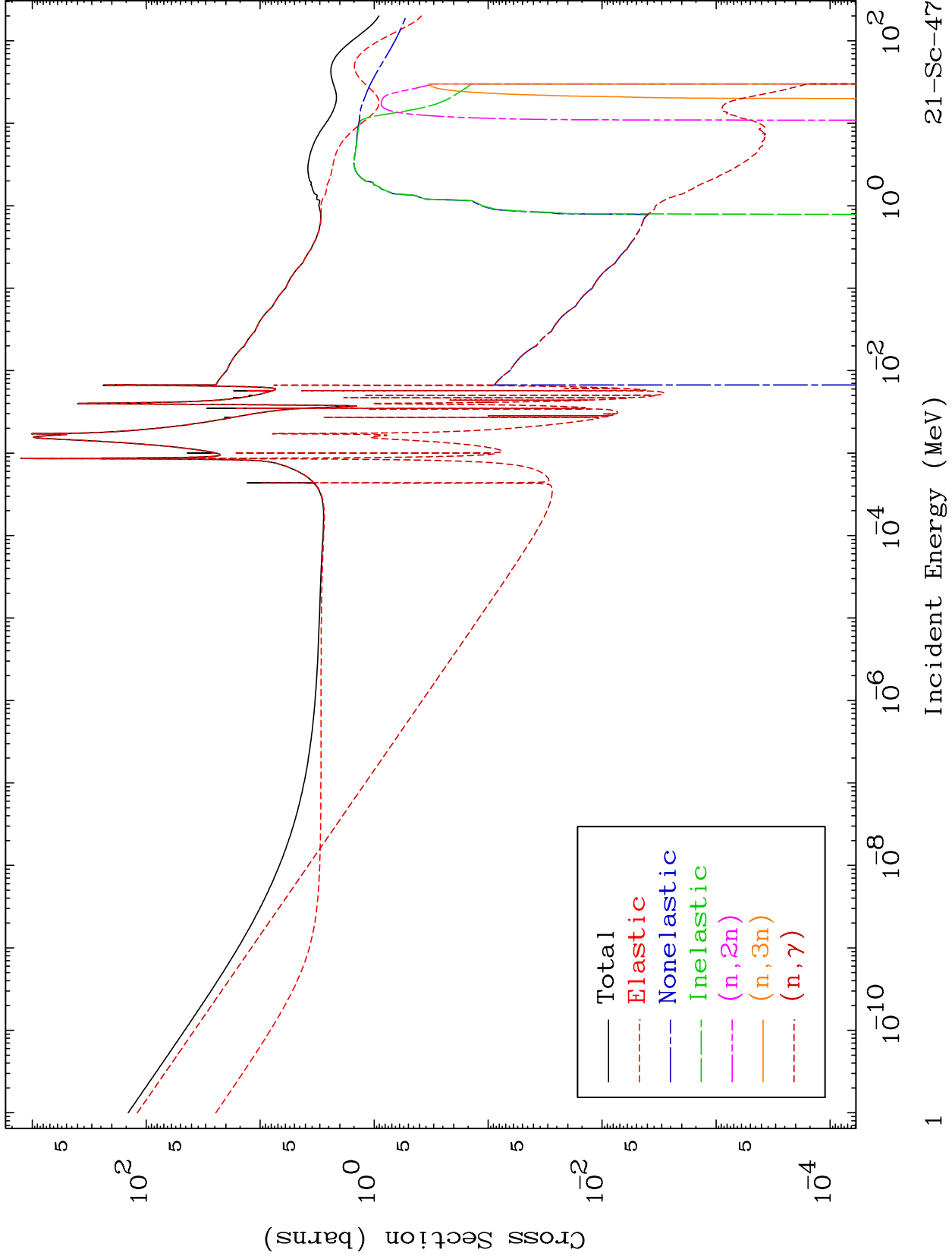
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Press Mouse Button to Start

MAT 2131

Neutron Major
293 Kelvin Cross Sections

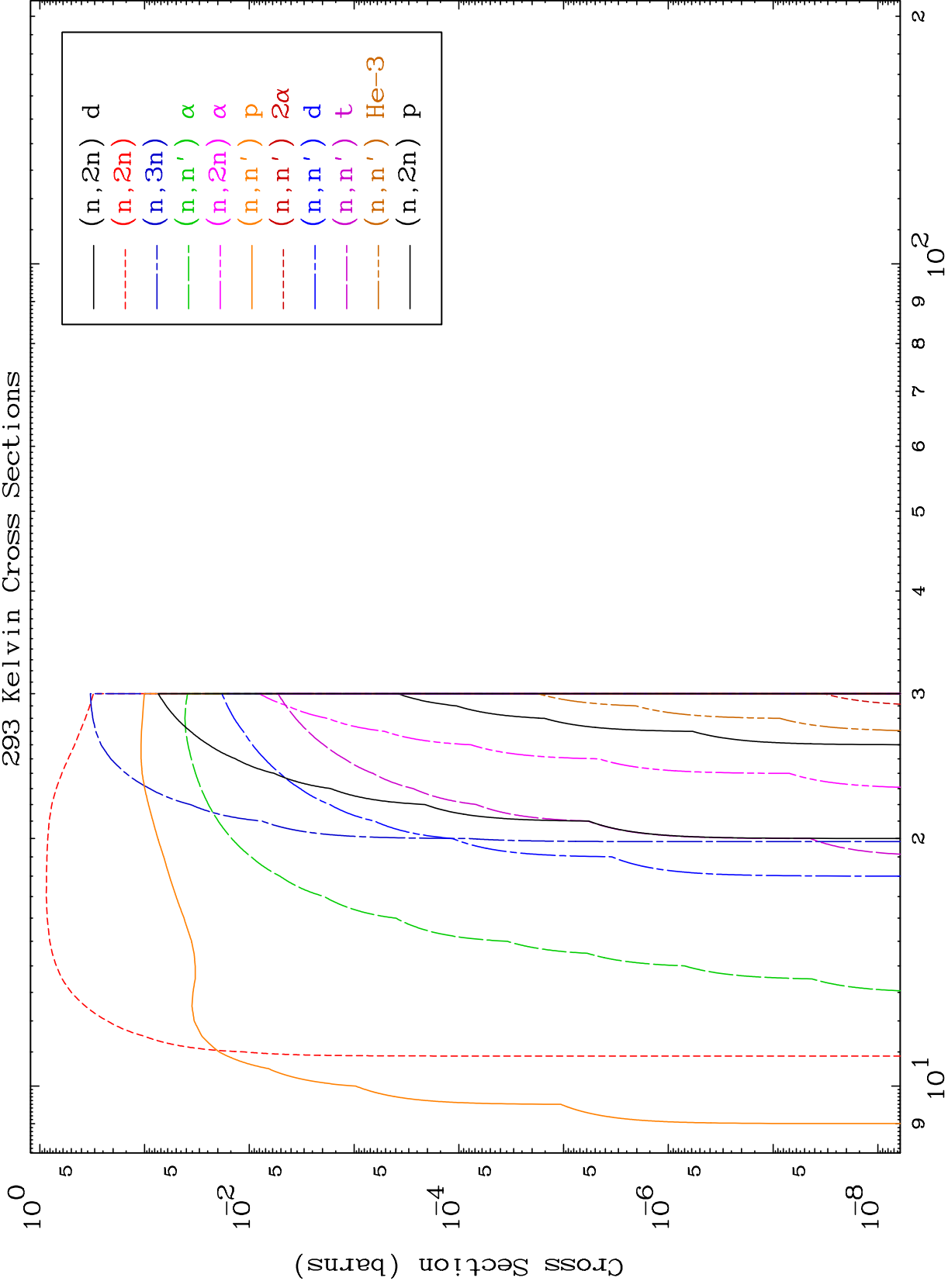
21-Sc-47



MAT 2131

Neutron Absorption
293 Kelvin Cross Sections

21-Sc-47



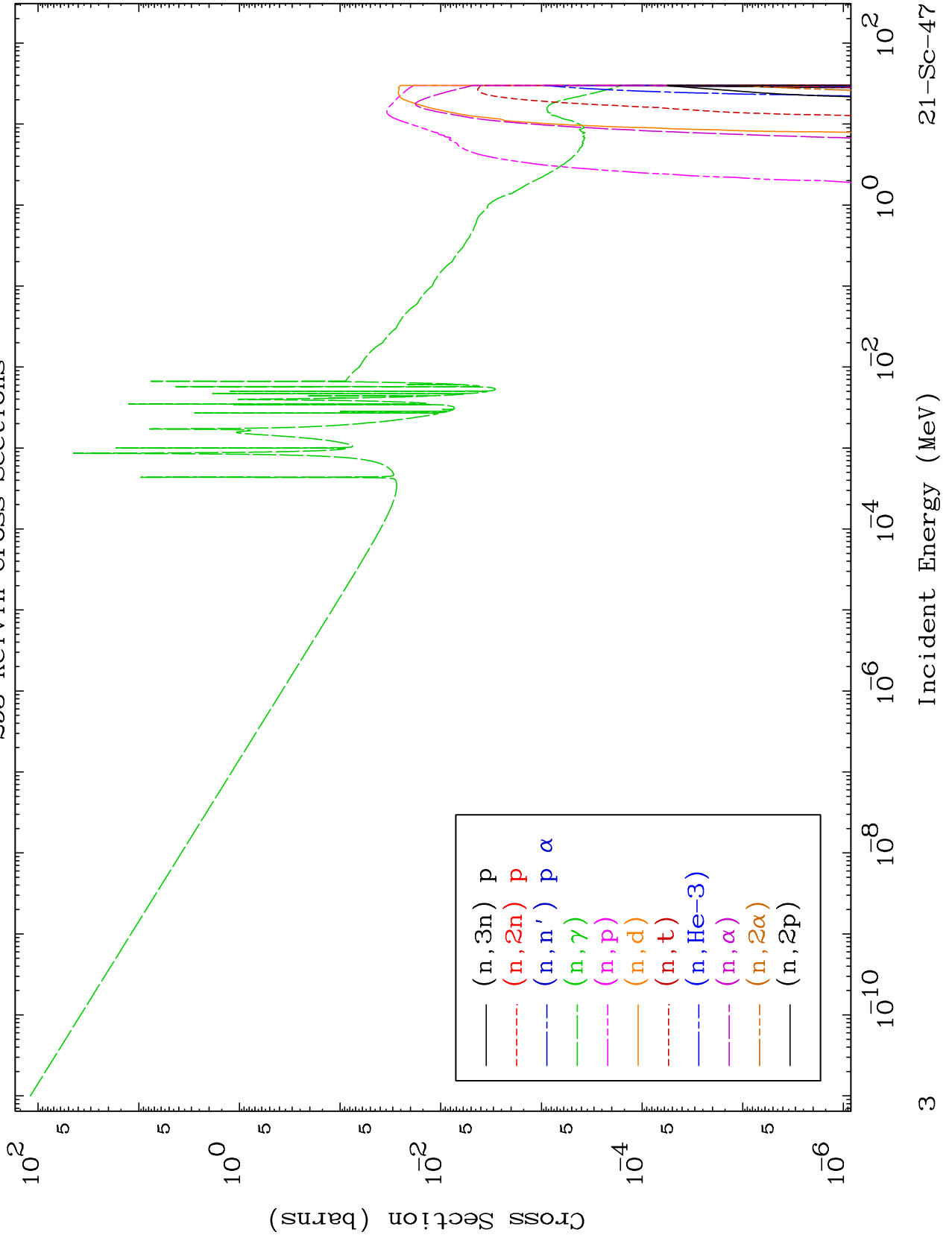
Incident Energy (MeV)

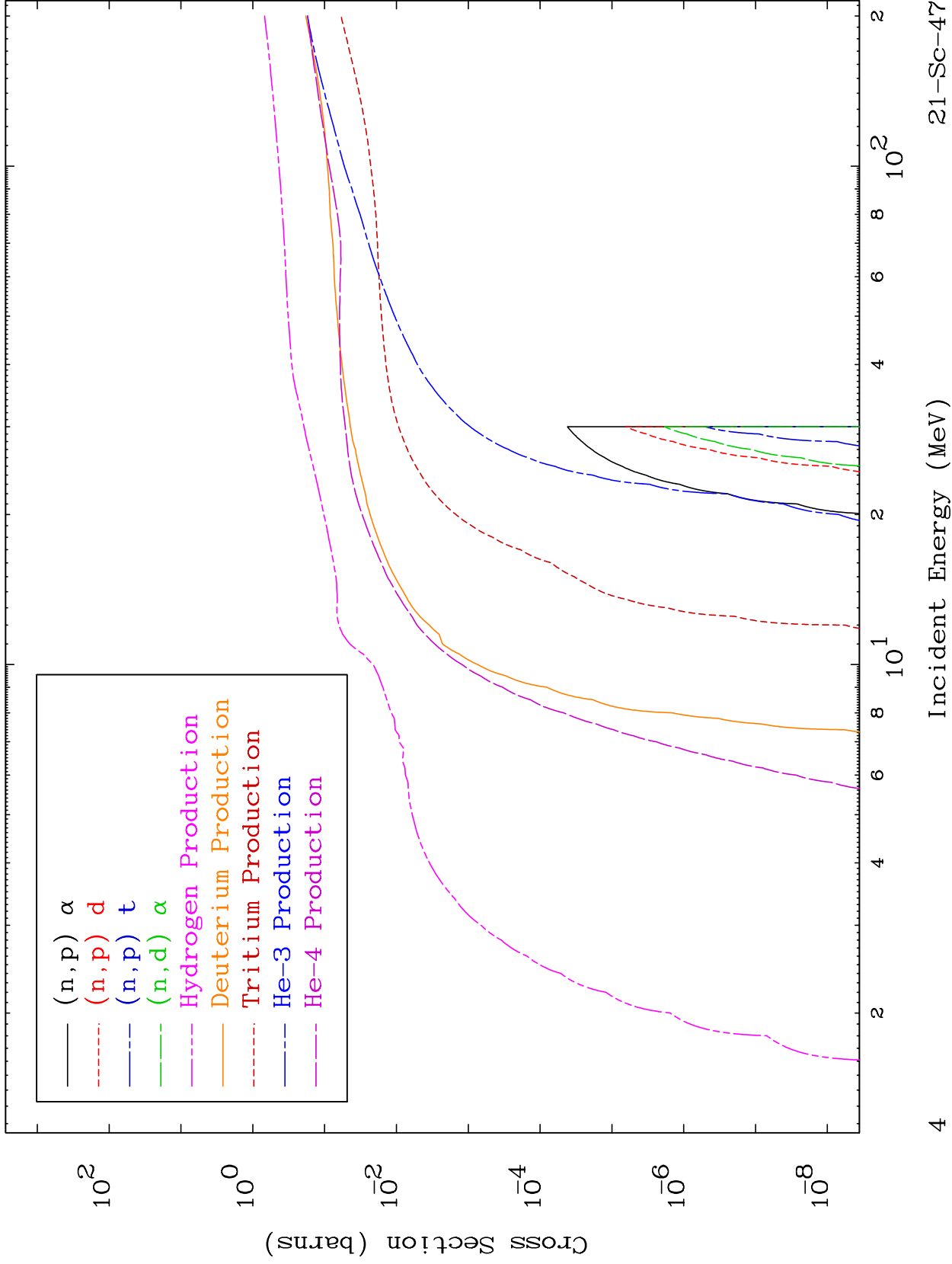
21-Sc-47

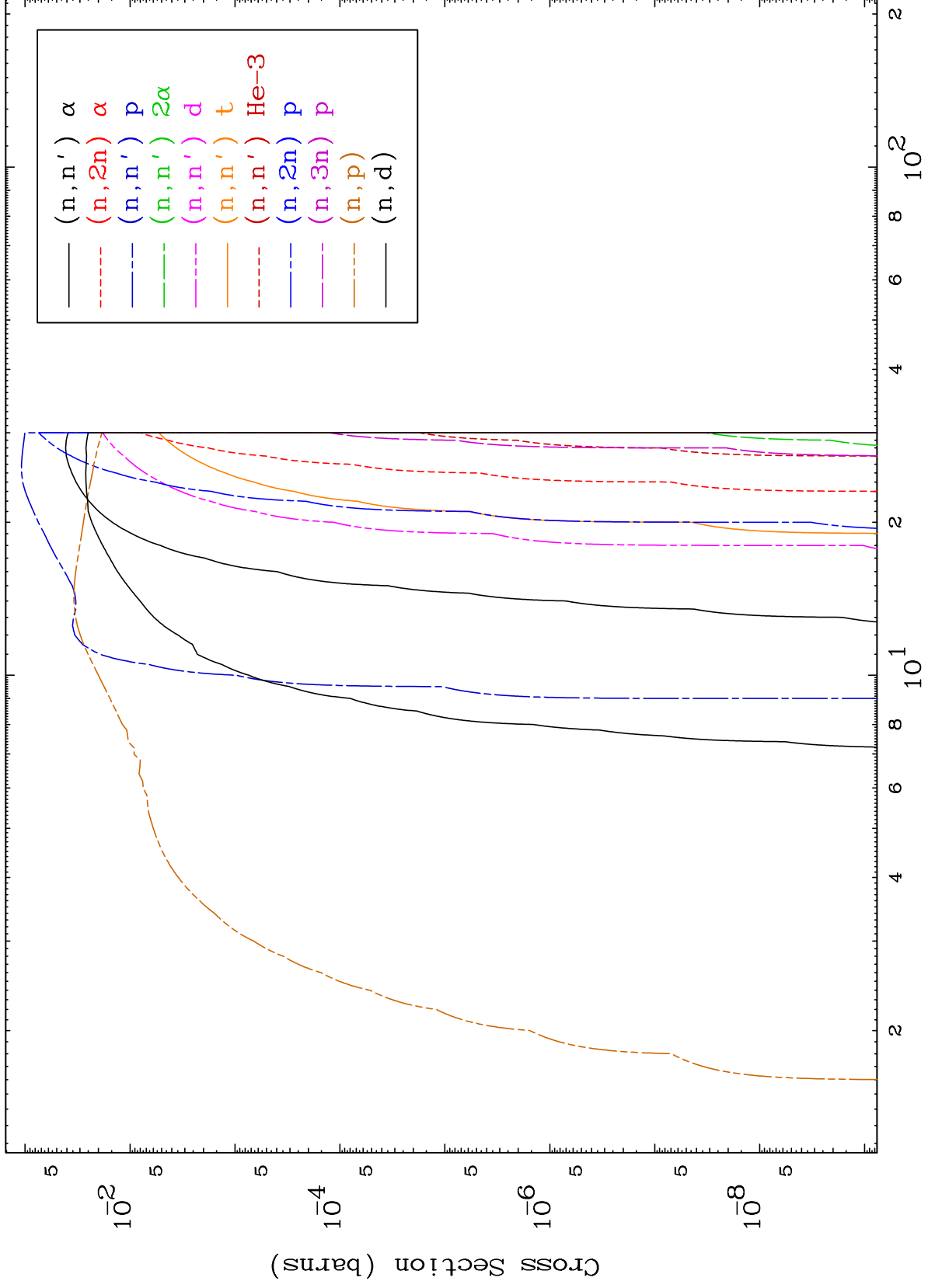
MAT 2131

Neutron Absorption
293 Kelvin Cross Sections

21-Sc-47



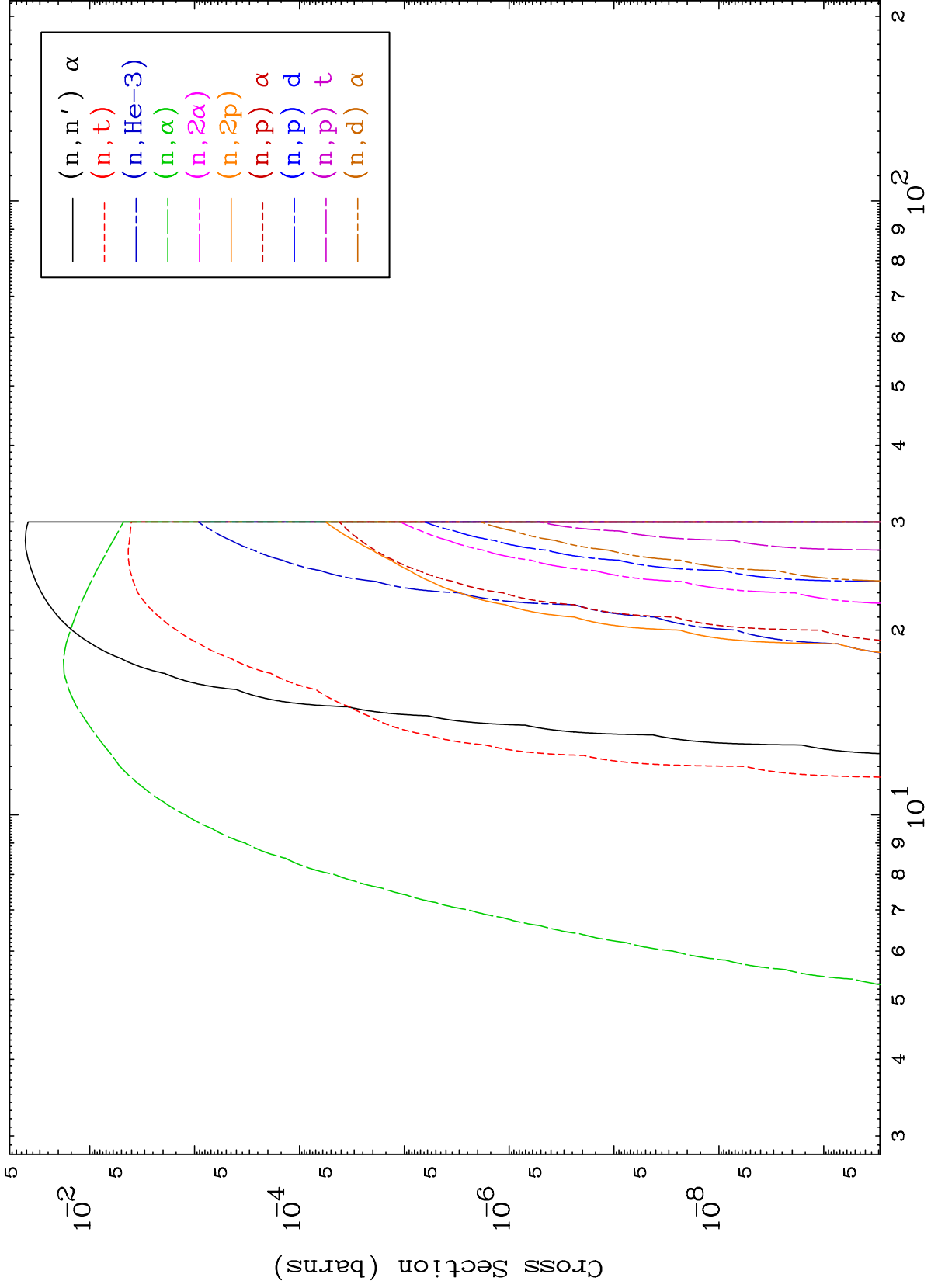




MAT 2131

Charged Particle
293 Kelvin Cross Sections

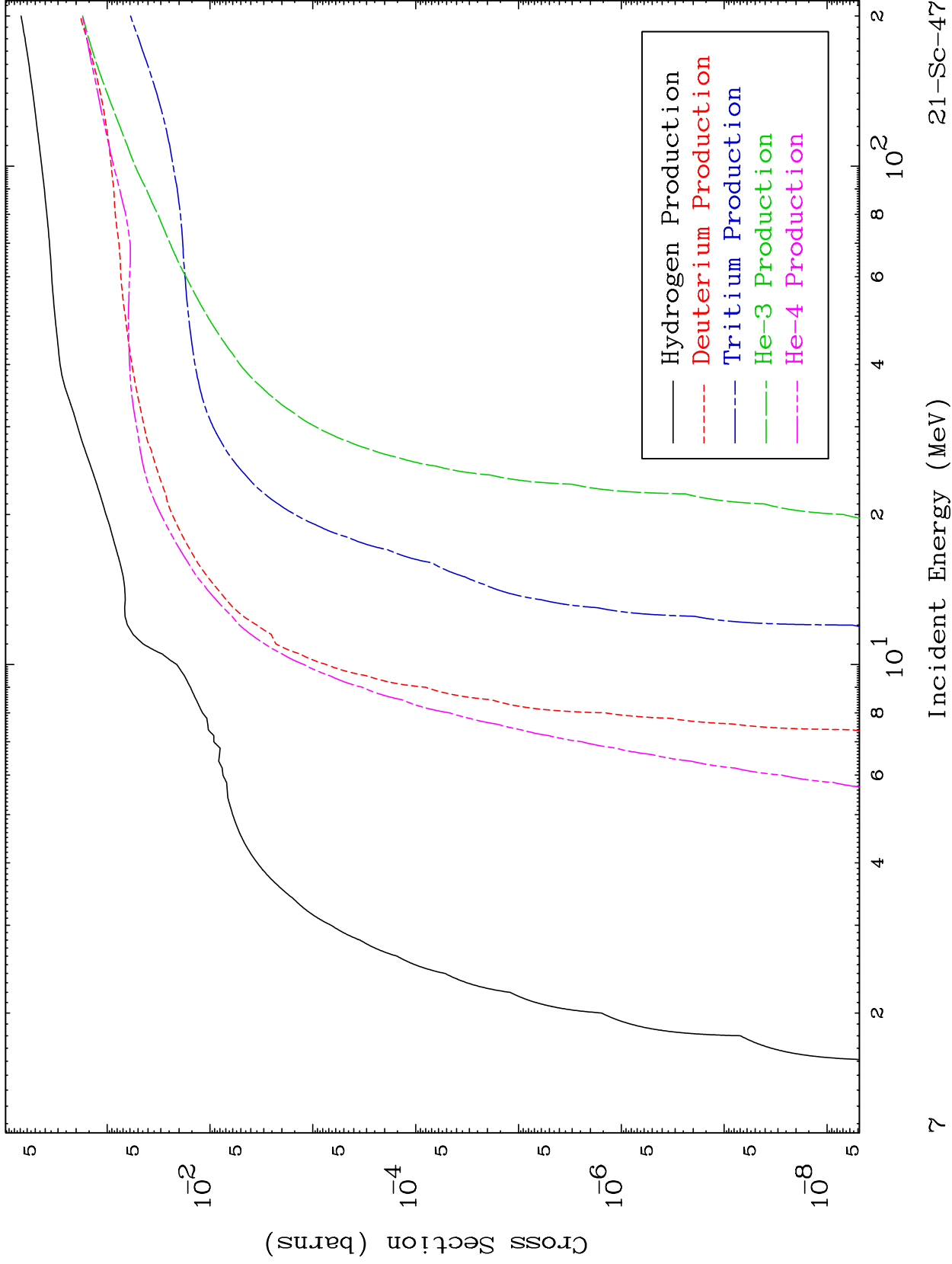
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Incident Energy (MeV)

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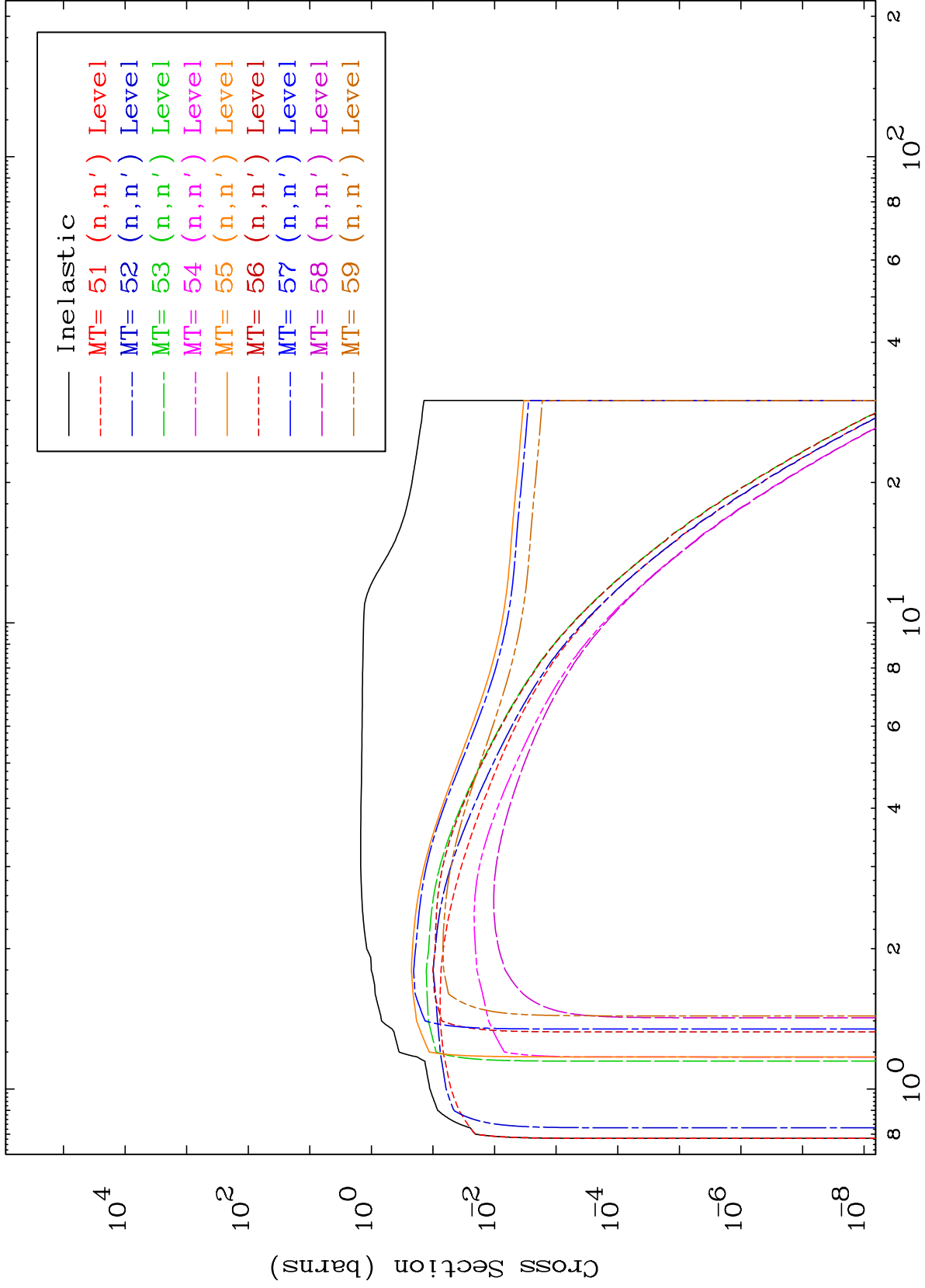


MAT 2131

(n,n') Levels

21-Sc-47

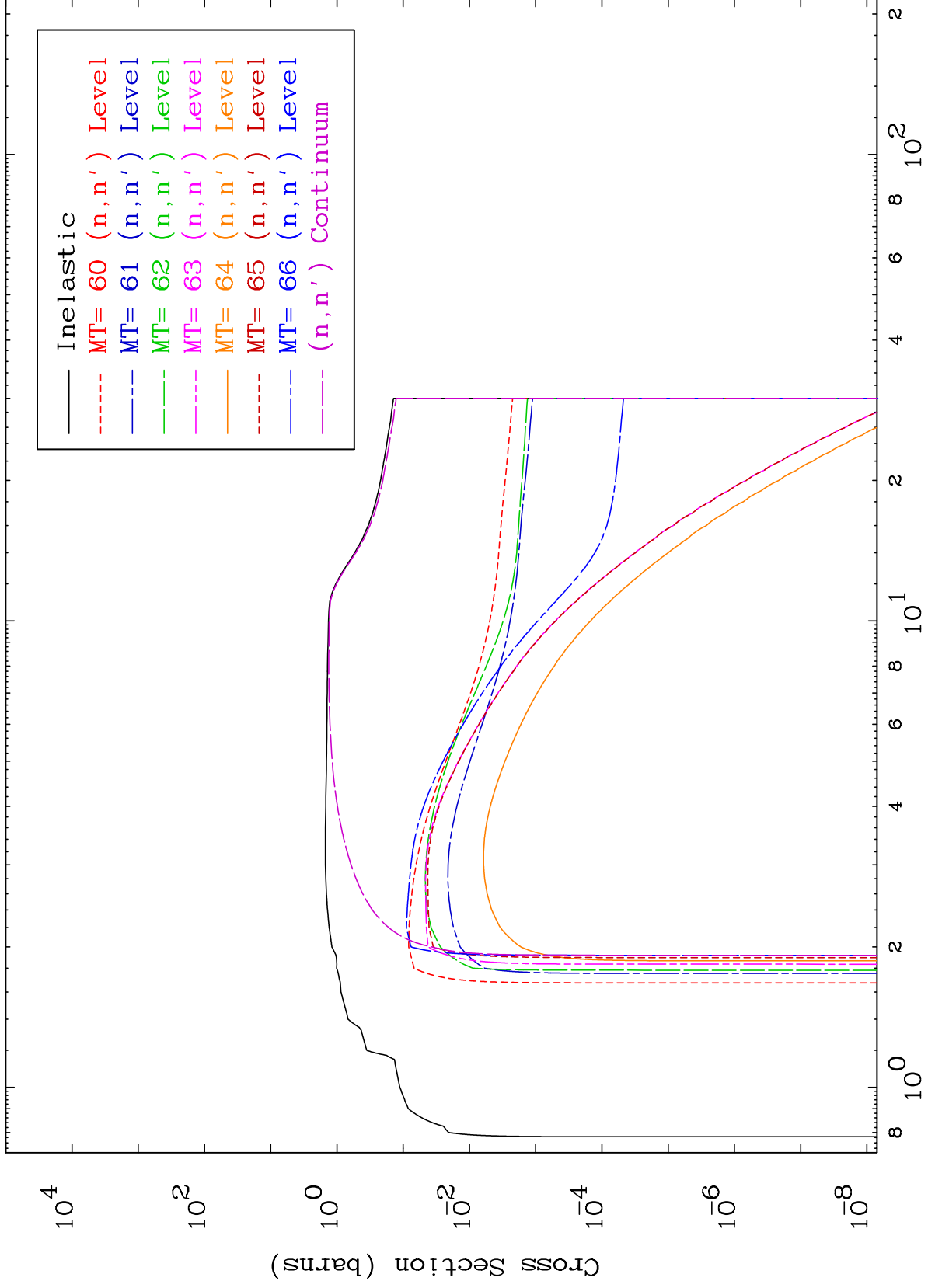
293 Kelvin Cross Sections



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Incident Energy (MeV)

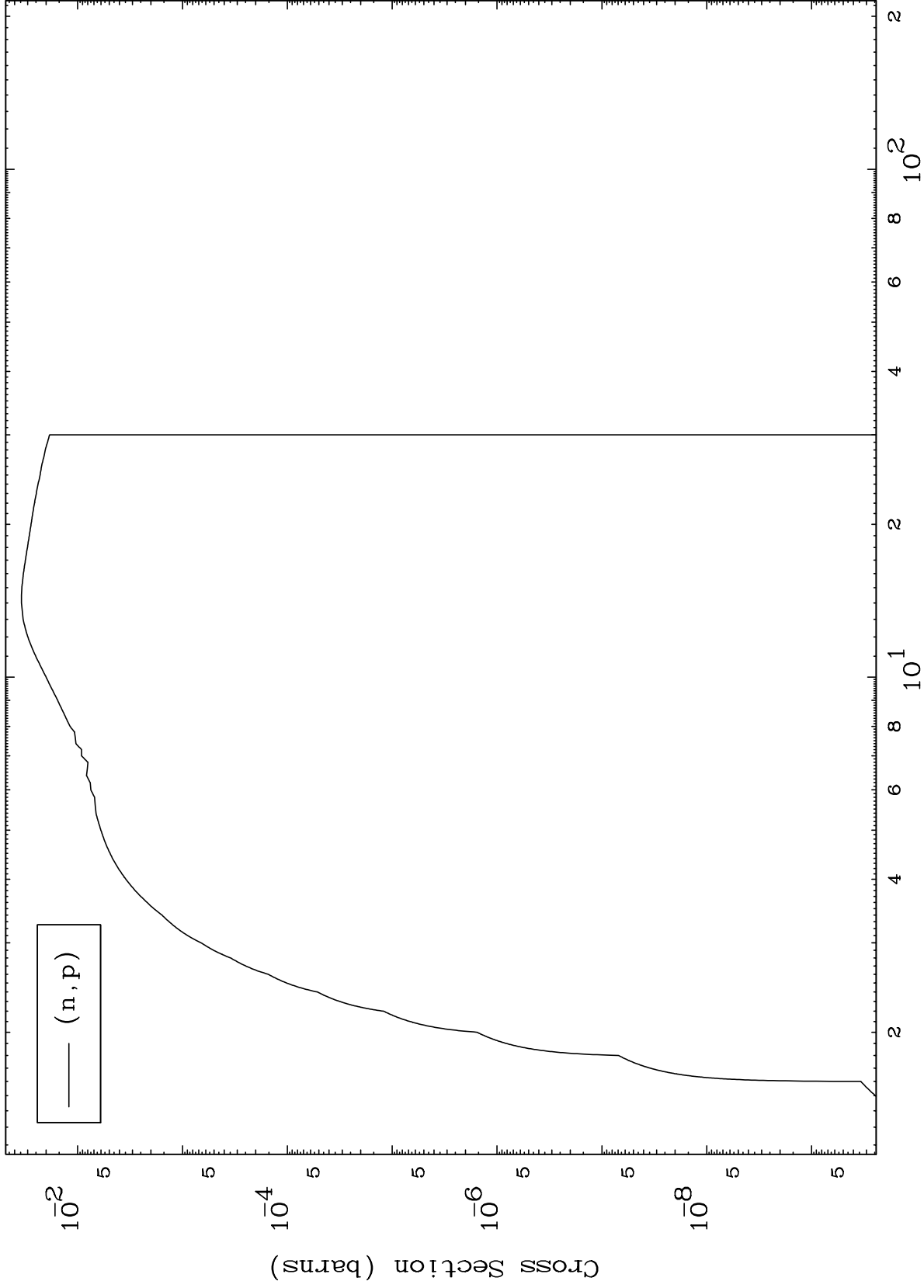
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MAT 2131

(n,p) Levels
293 Kelvin Cross Sections

21-Sc-47



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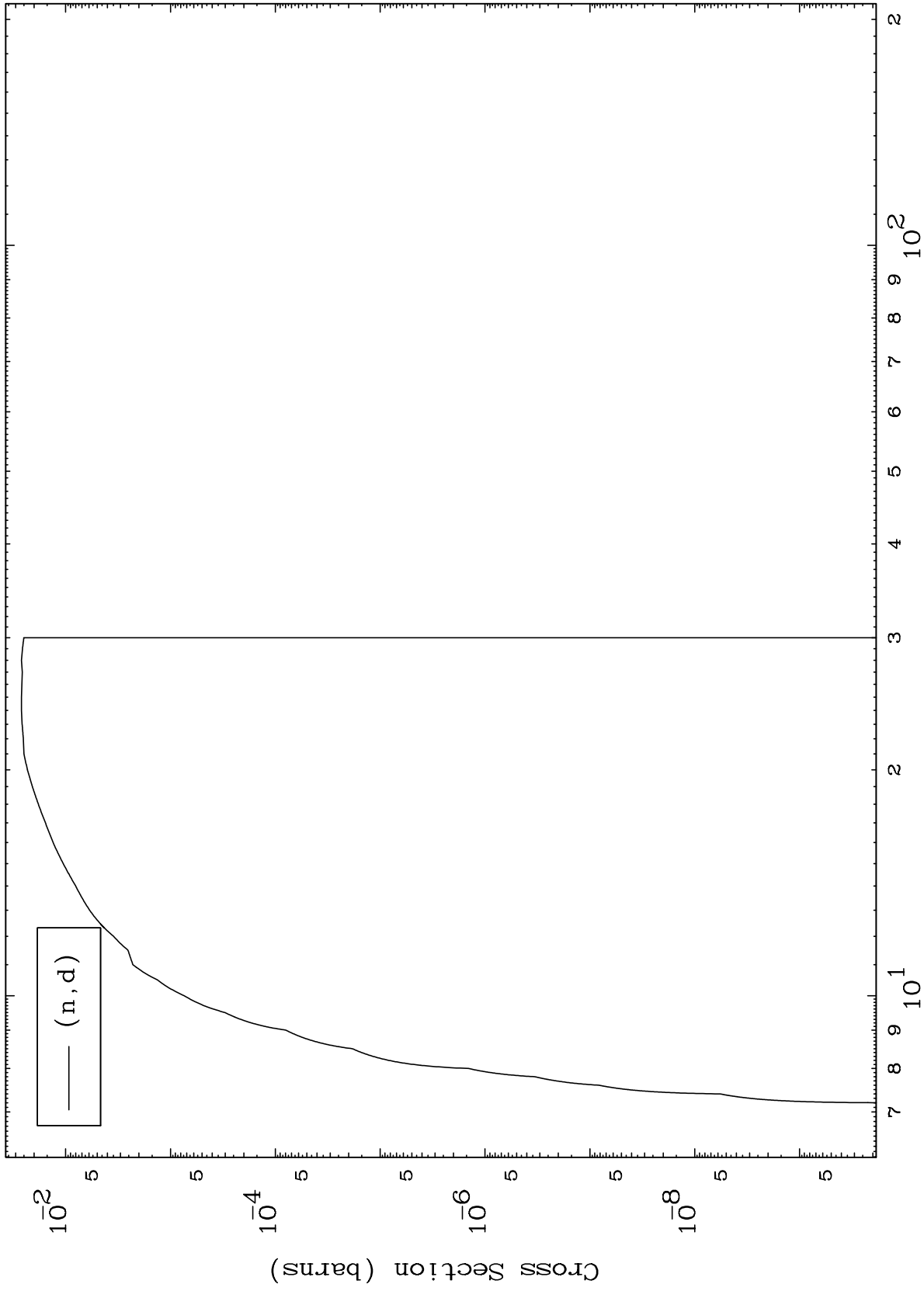
Incident Energy (MeV)

21-Sc-47

MAT 2131

(n,d) Levels
293 Kelvin Cross Sections

21-Sc-47



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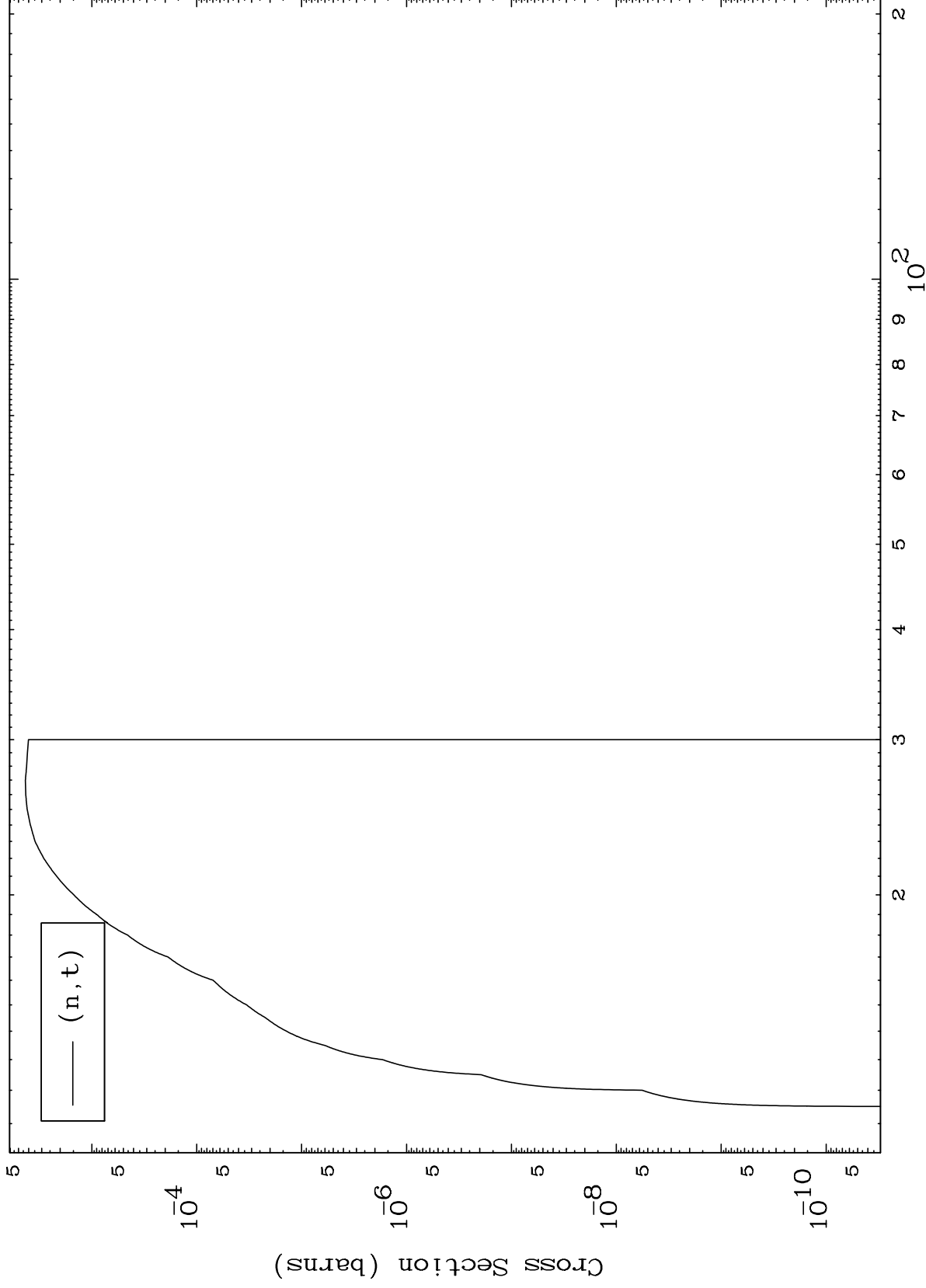
Incident Energy (MeV)

21-Sc-47

MAT 2131

(n,t) Levels
293 Kelvin Cross Sections

21-Sc-47



12

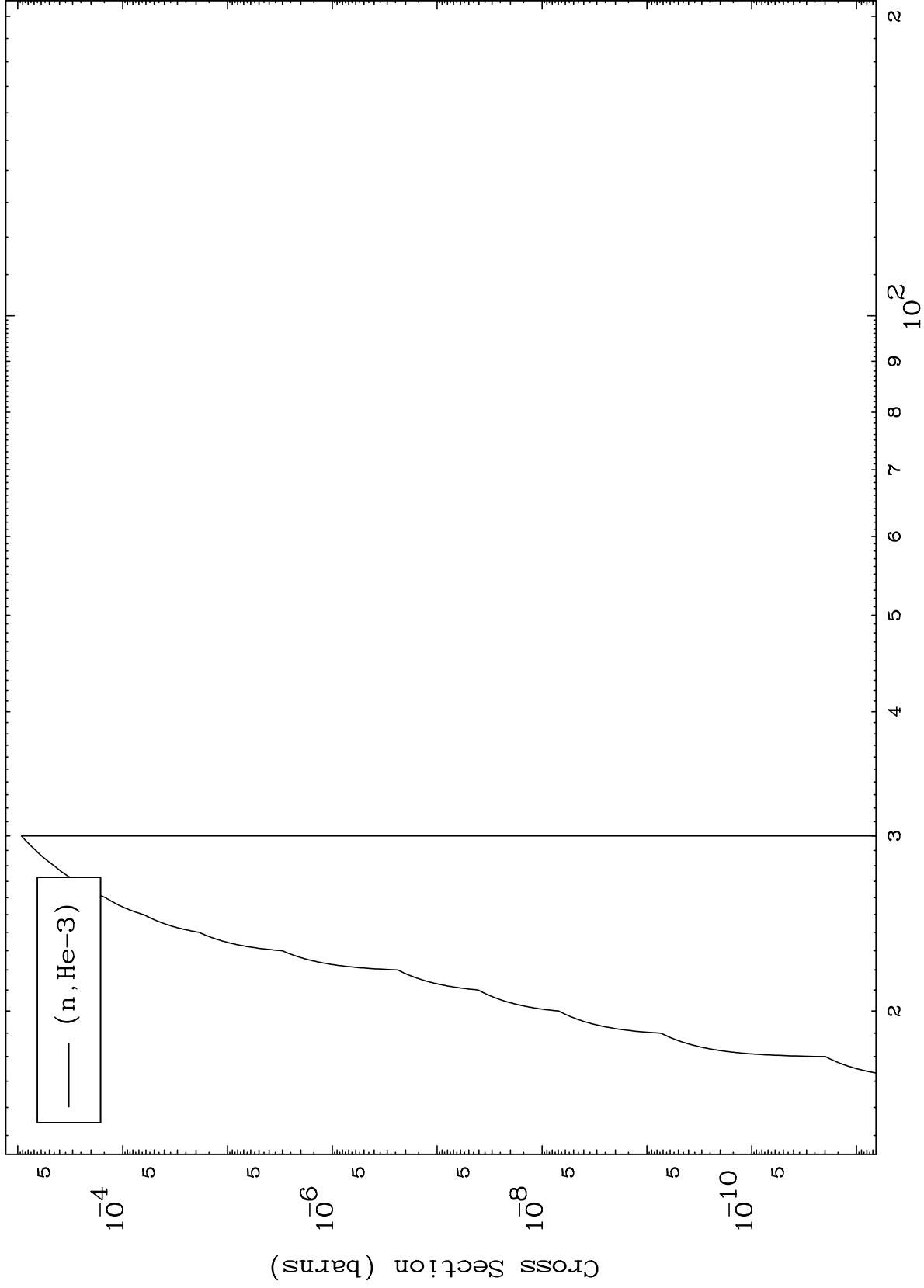
Incident Energy (MeV)

21-Sc-47

MAT 2131

(n,He3) Levels
293 Kelvin Cross Sections

21-Sc-47



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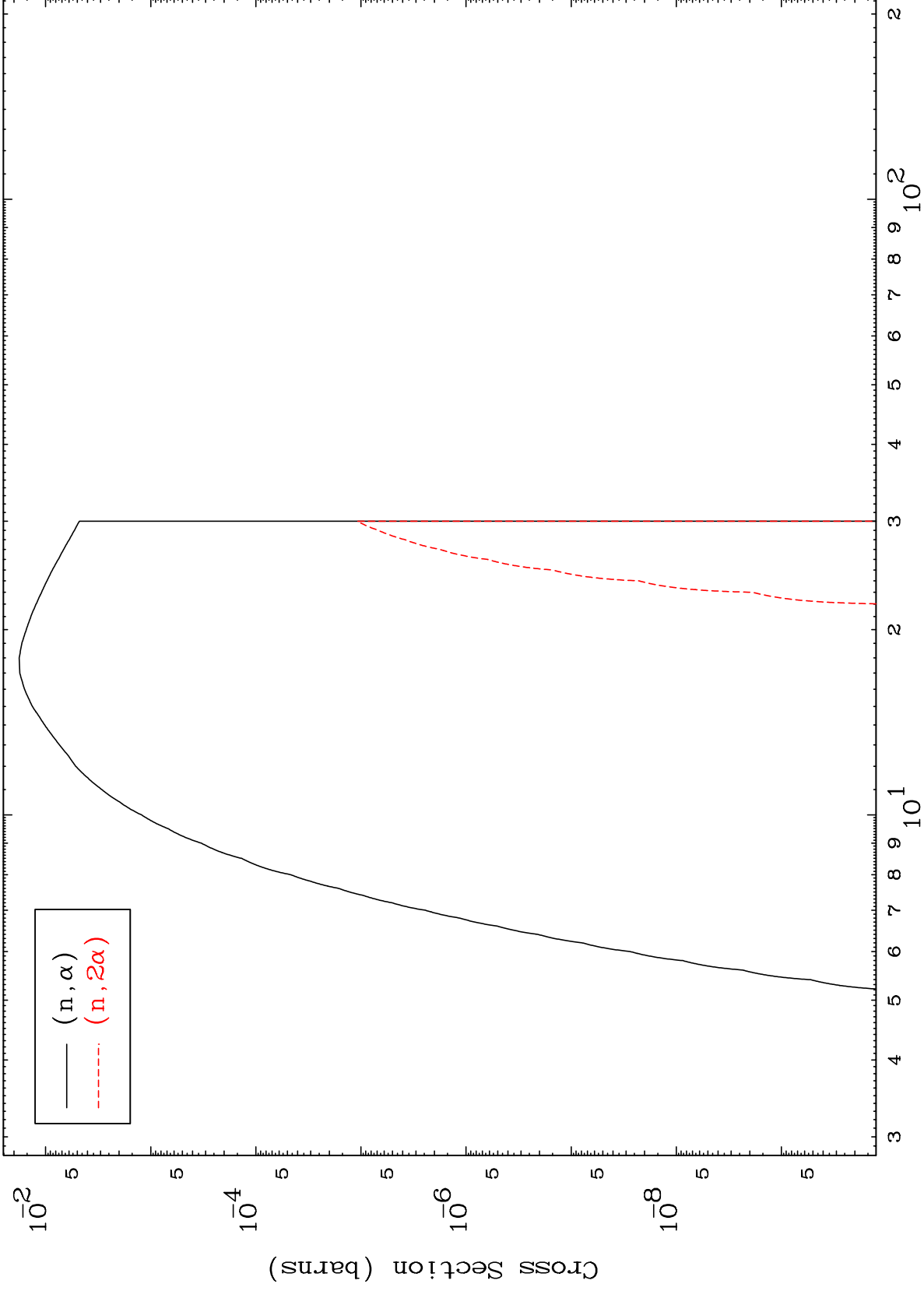
Incident Energy (MeV)

21-Sc-47

MAT 2131

(n, α) Levels
293 Kelvin Cross Sections

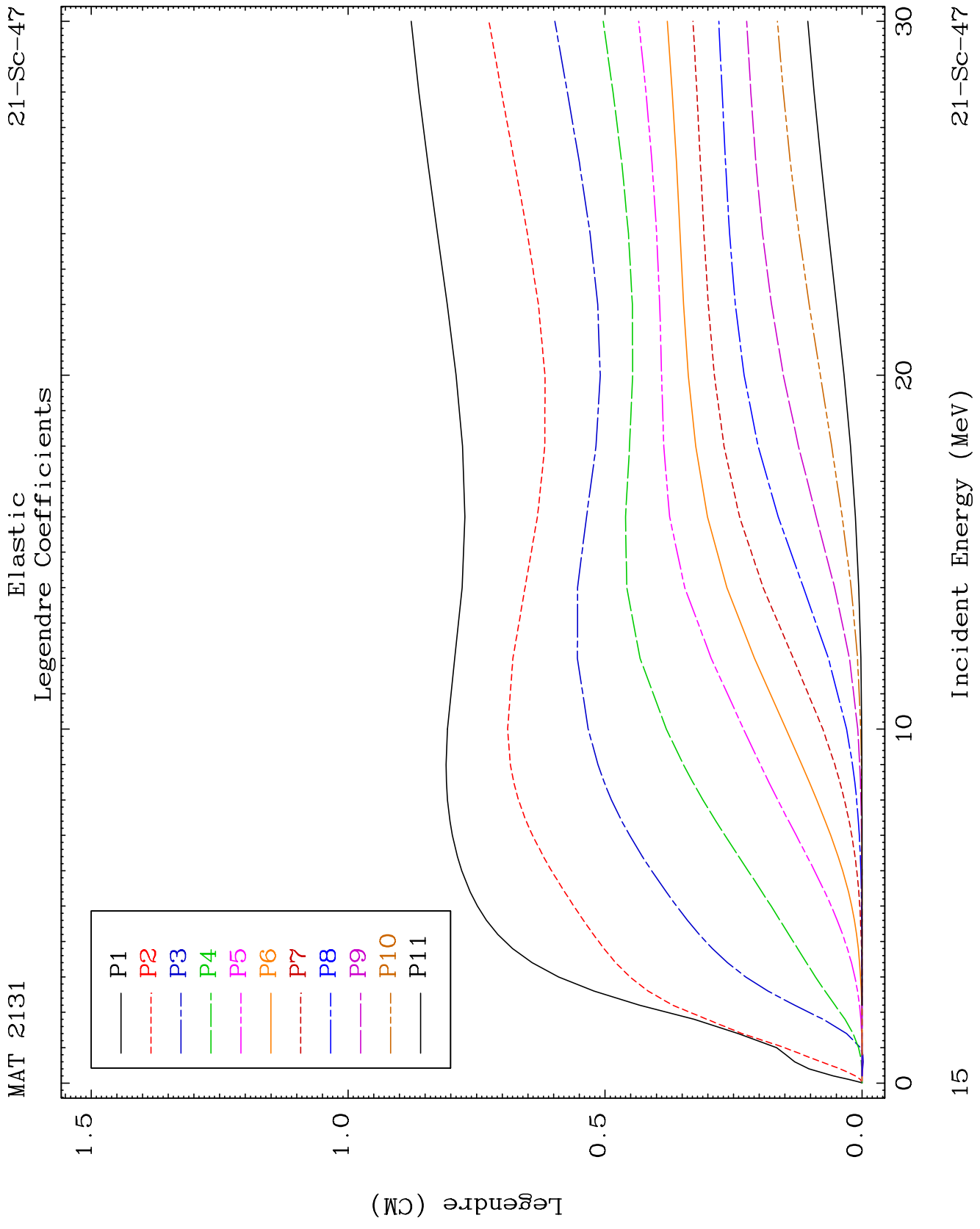
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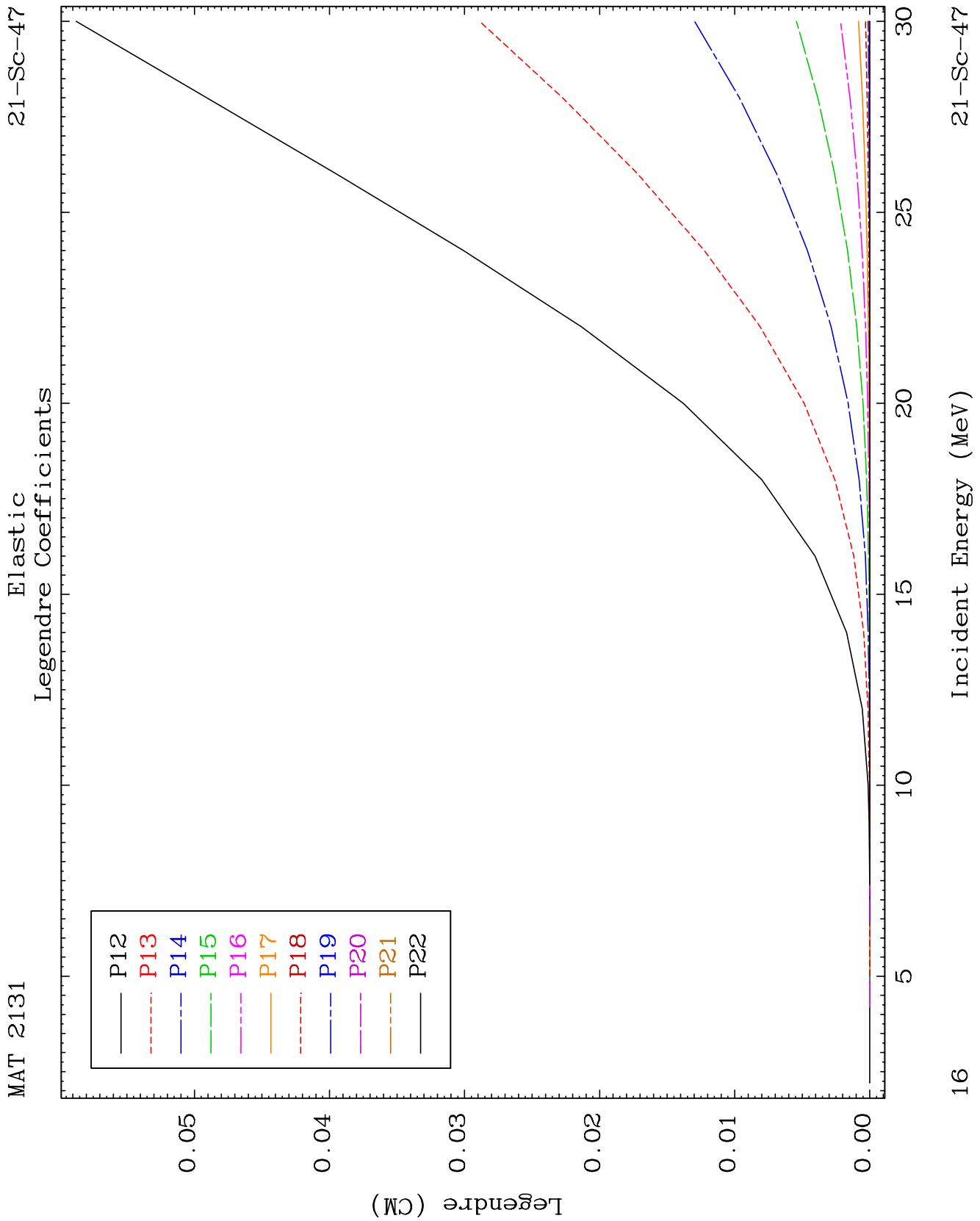


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Incident Energy (MeV)

21-Sc-47

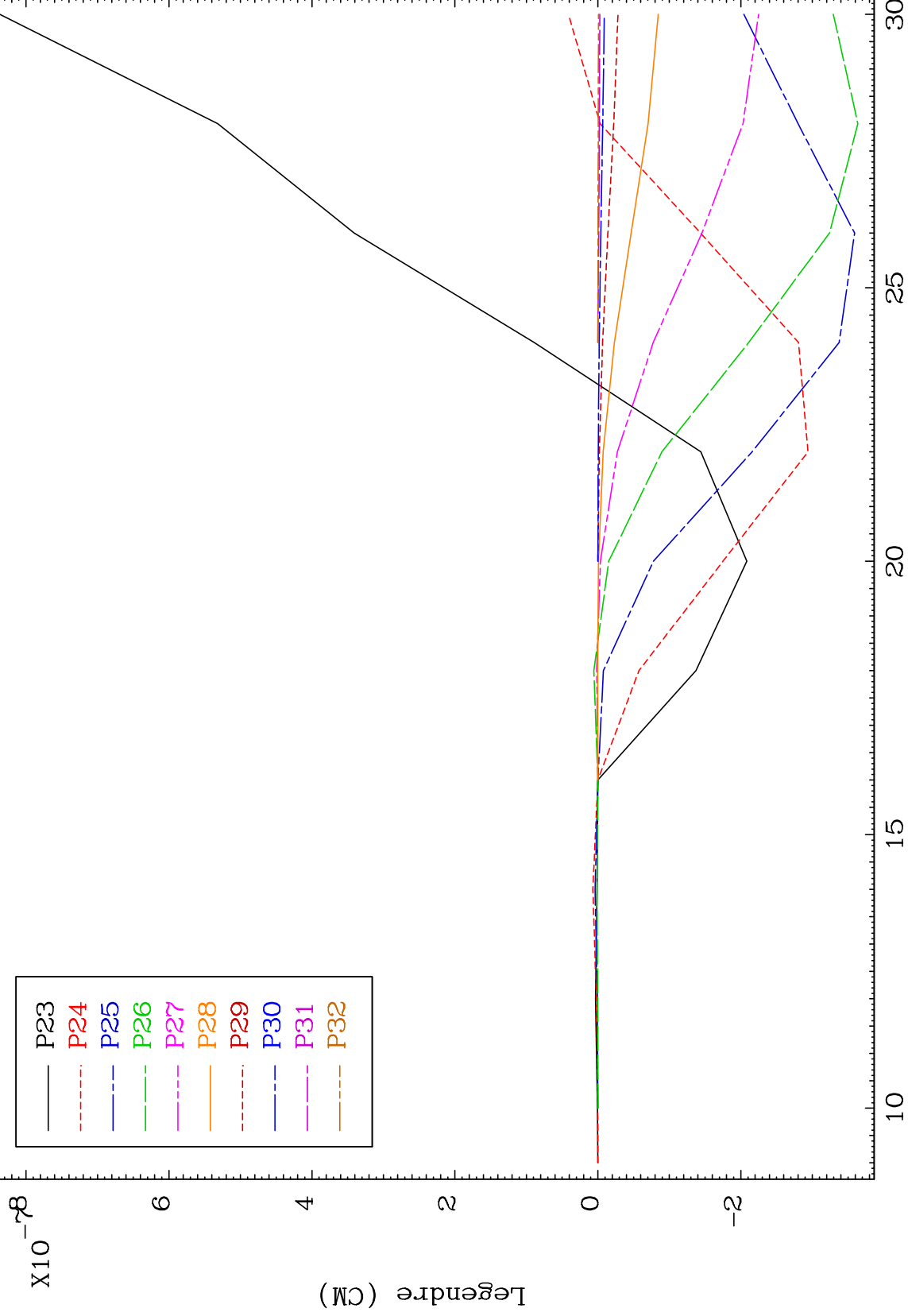




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Elastic Legendre Coefficients

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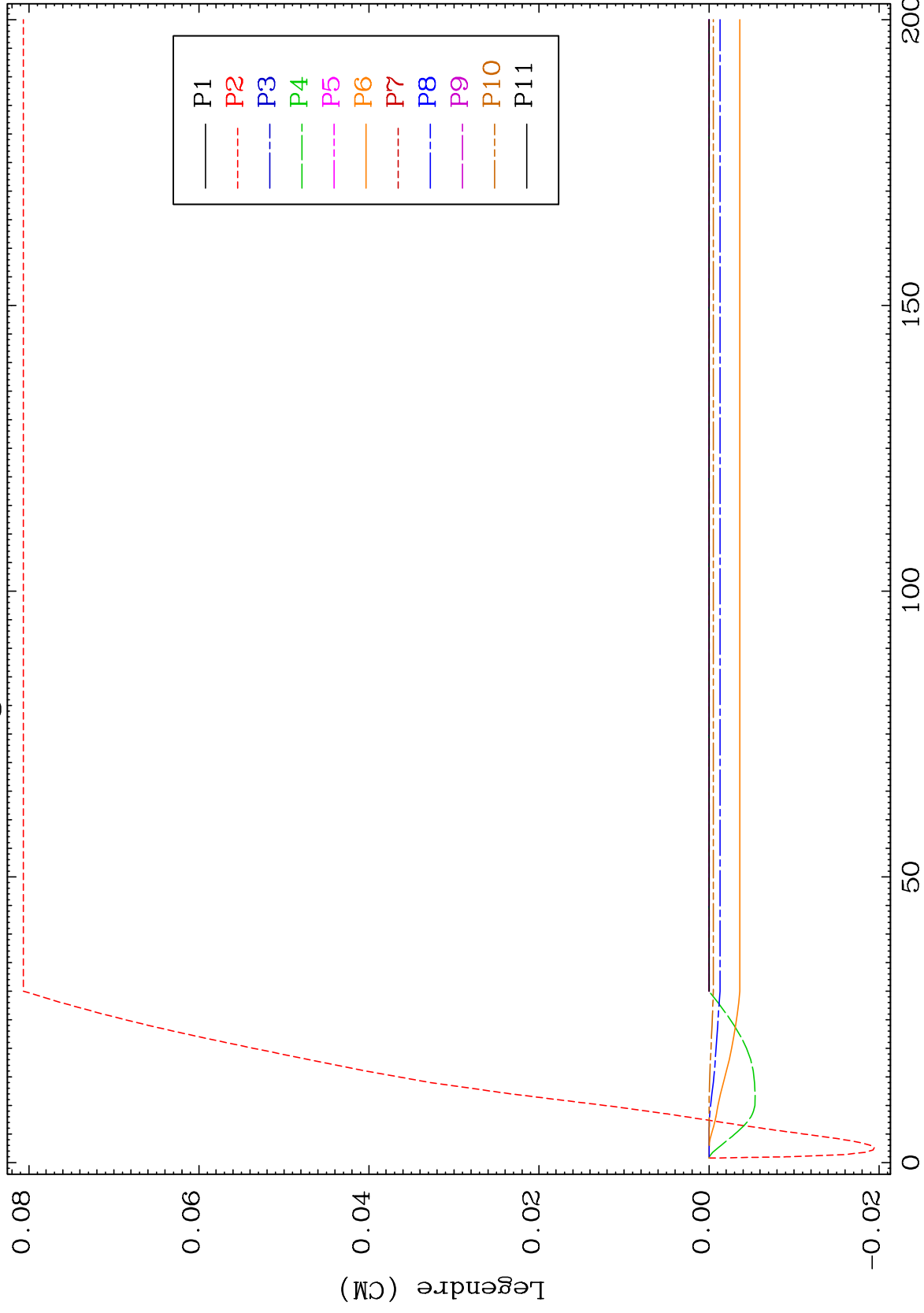
Incident Energy (MeV)

21-Sc-47

MAT 2131

MT= 51 (n,n') Level
Legendre Coefficients

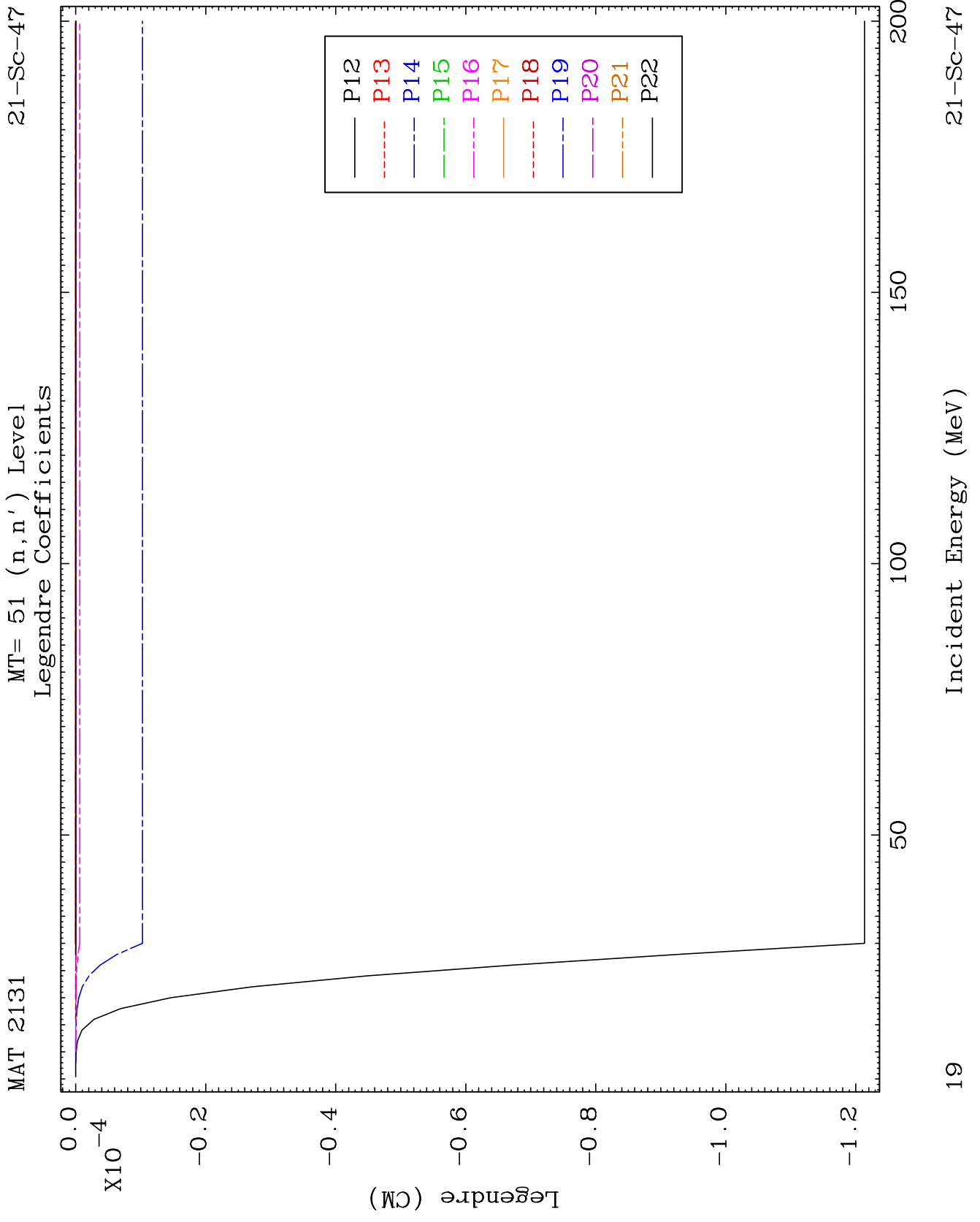
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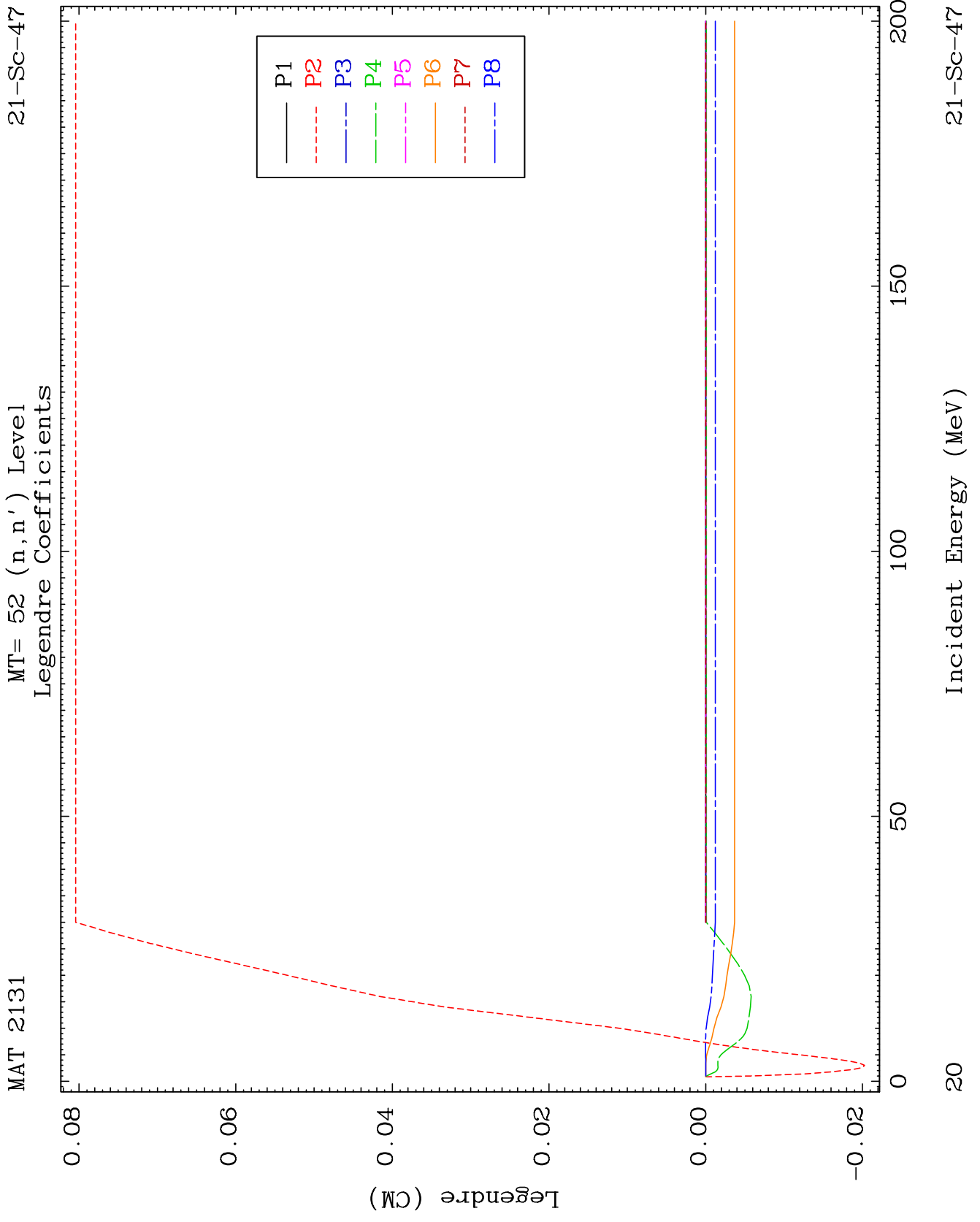


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Incident Energy (MeV)

21-Sc-47

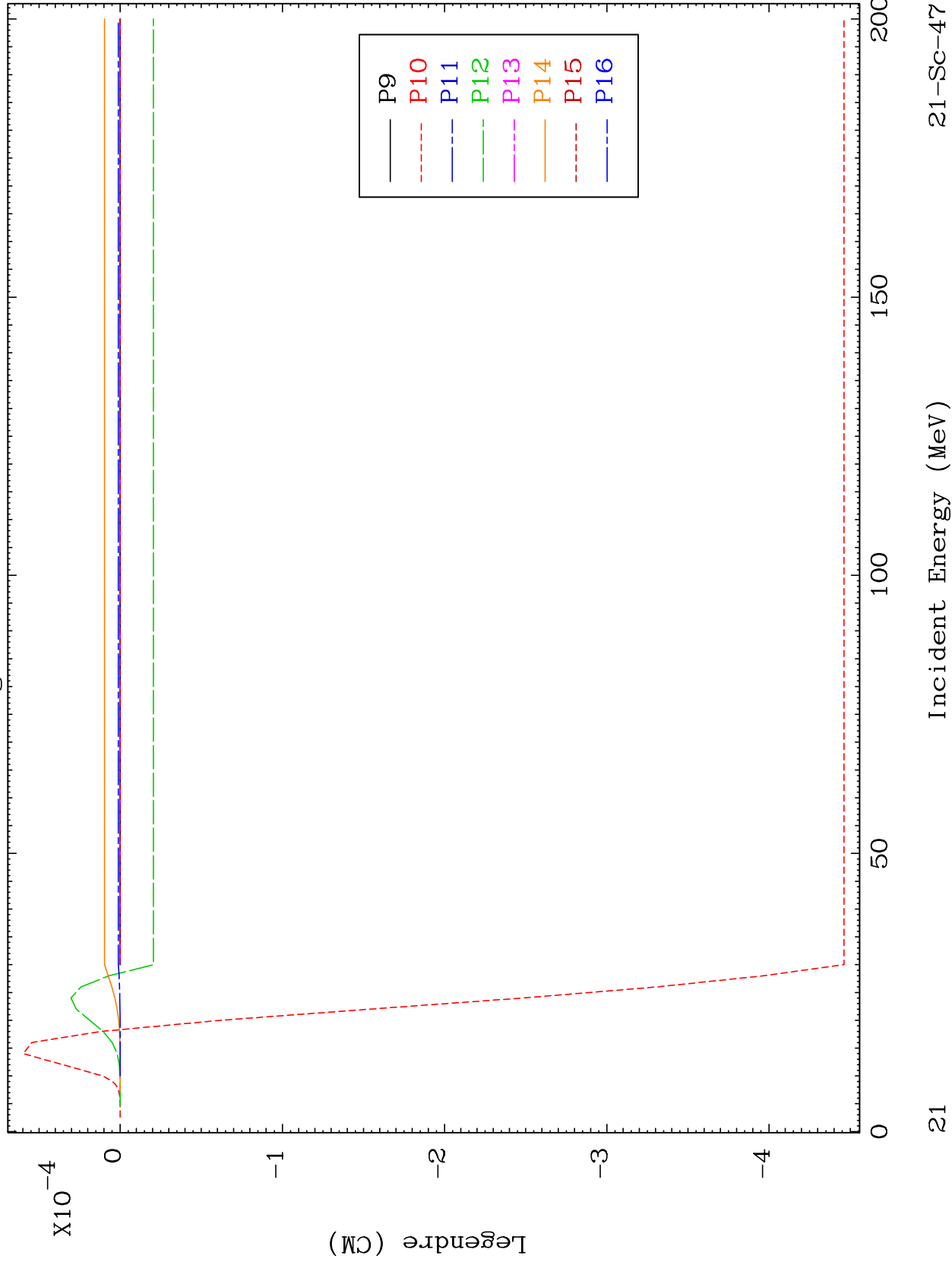


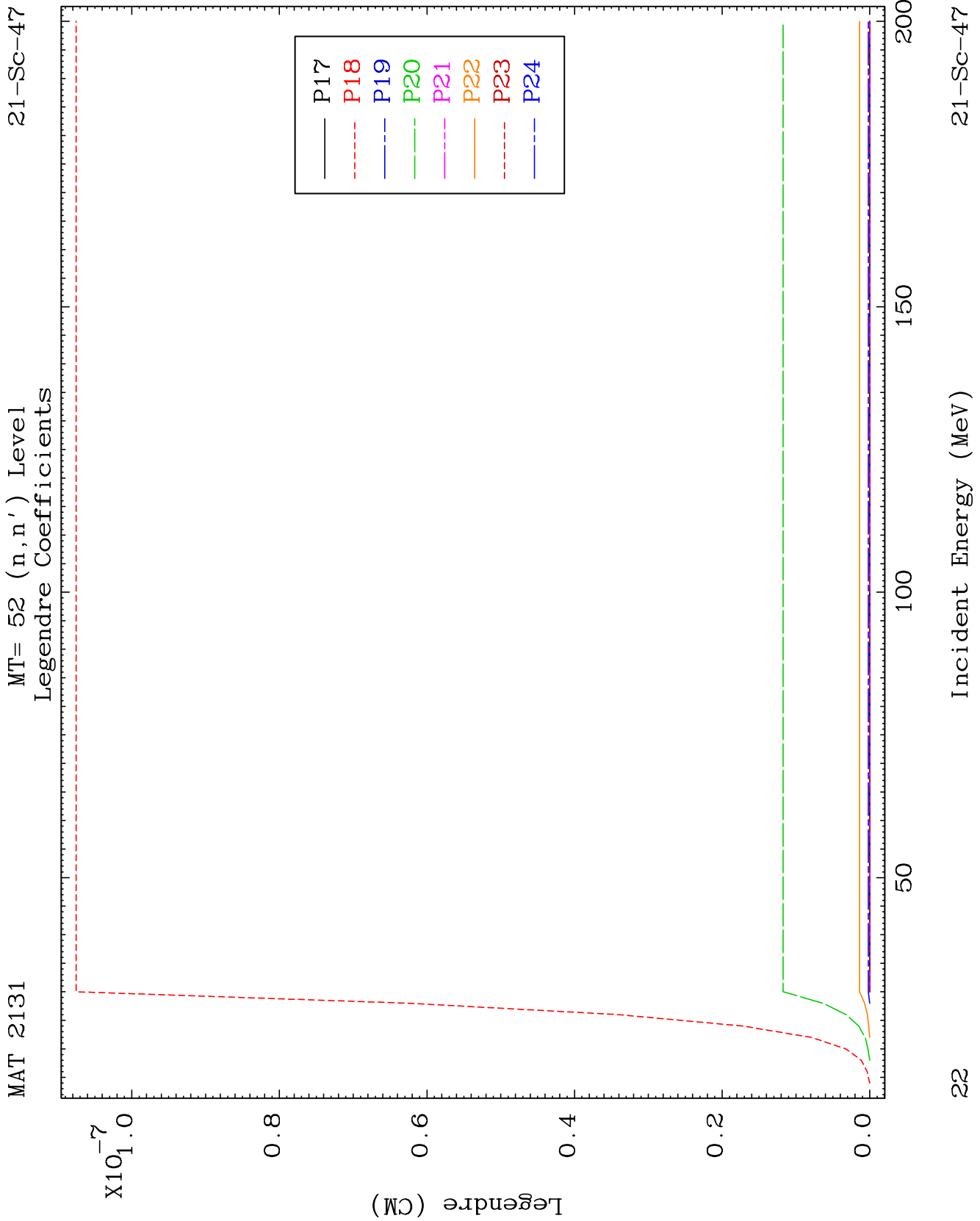


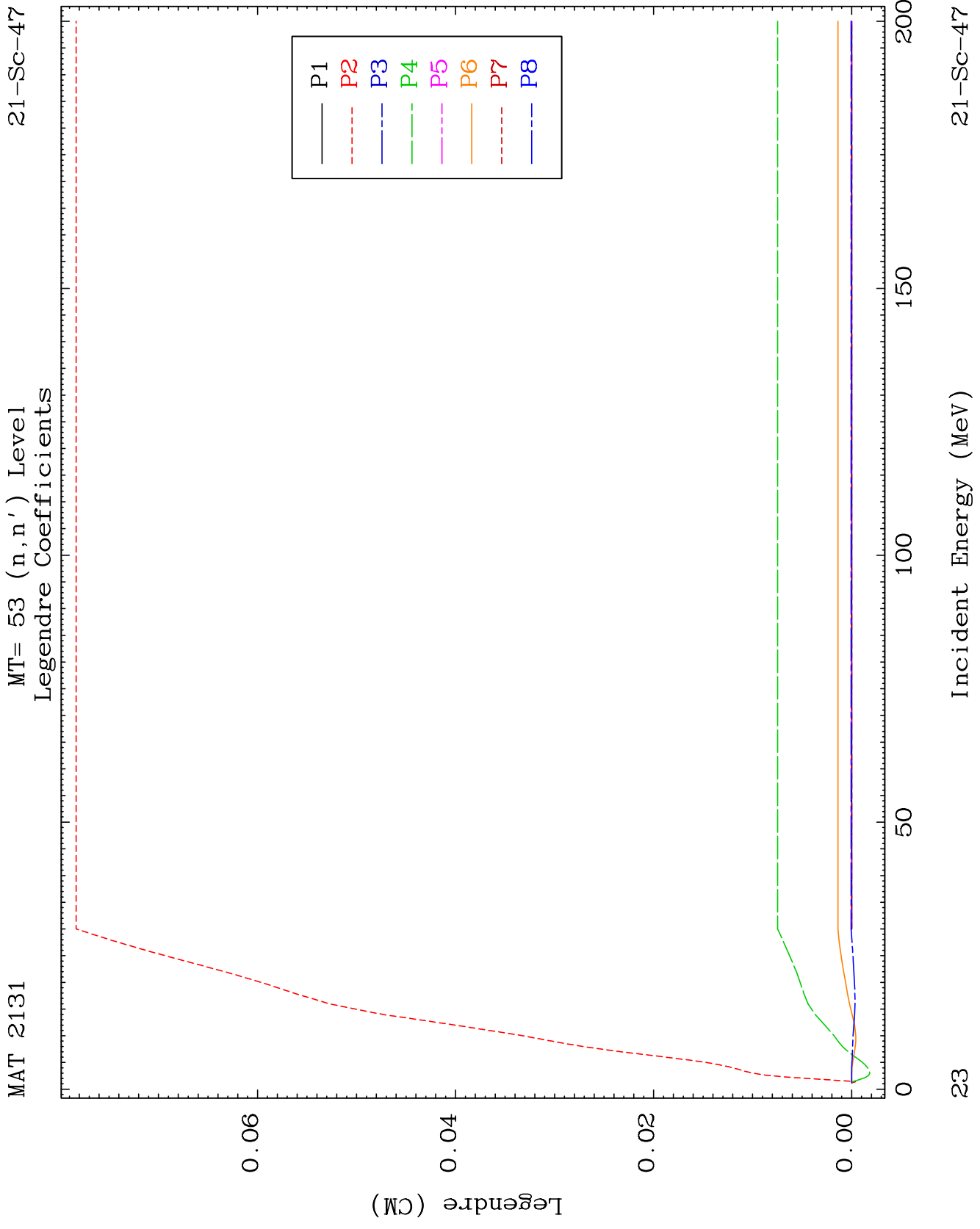
MAT 2131

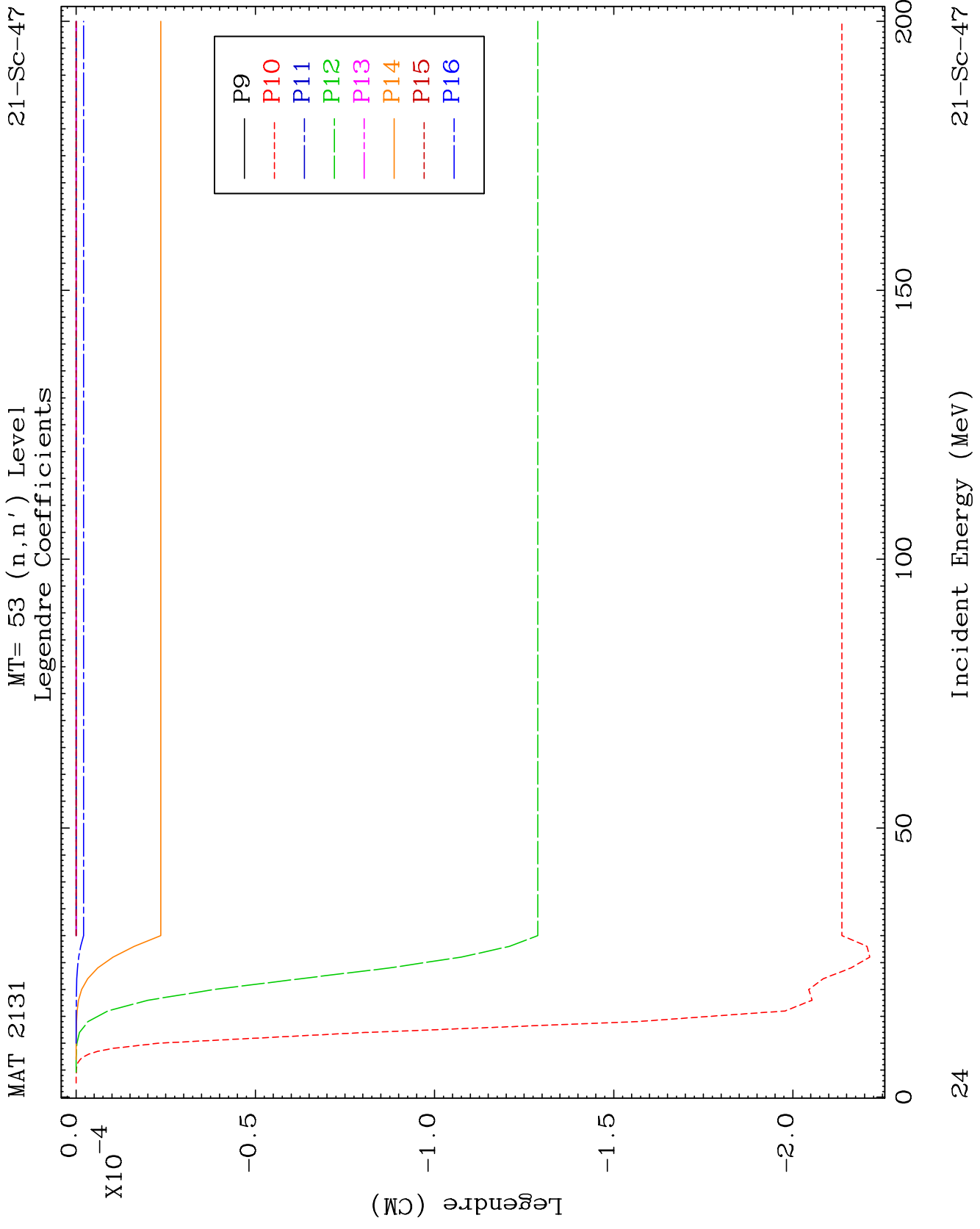
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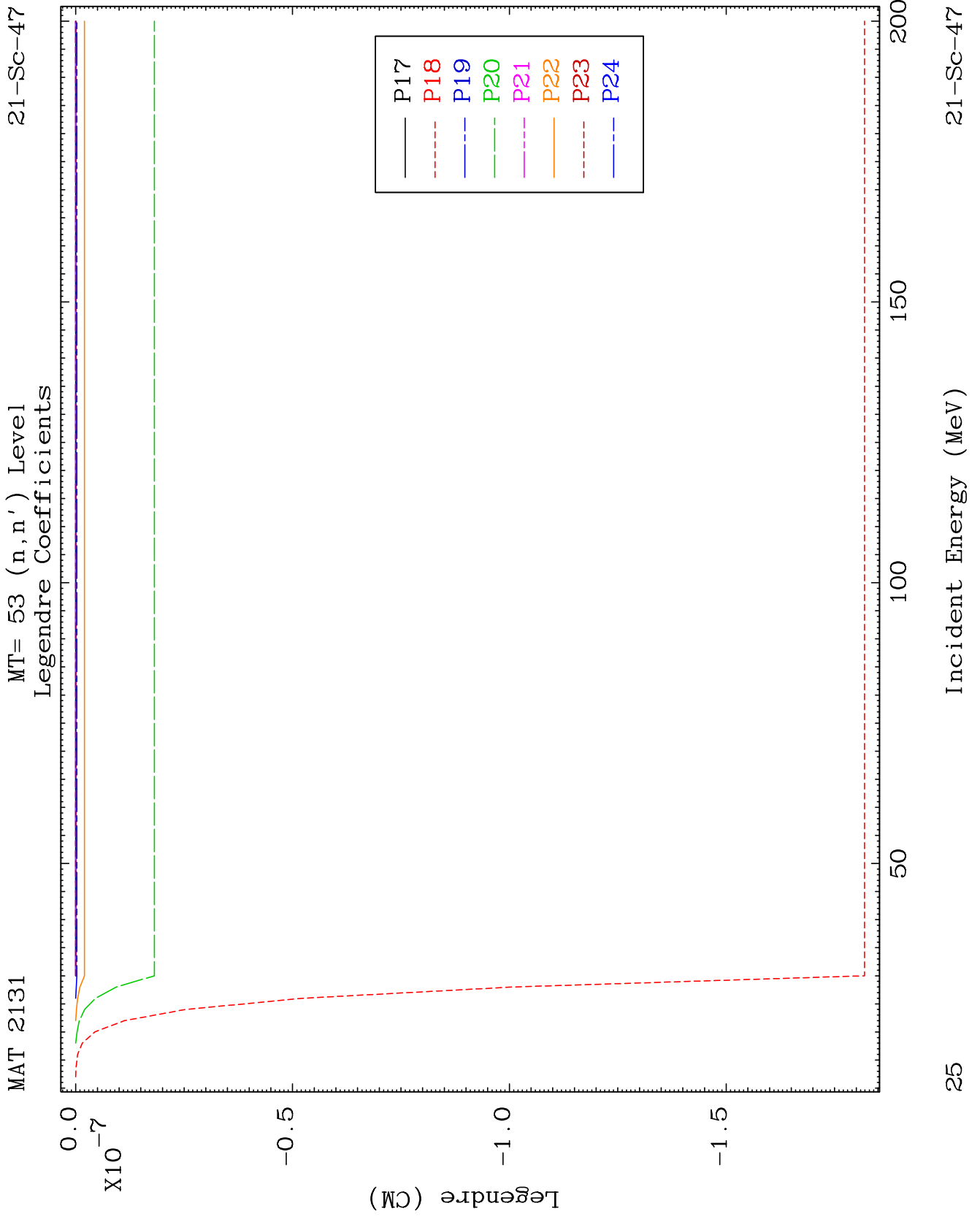
MT= 52 (n,n') Level
Legendre Coefficients

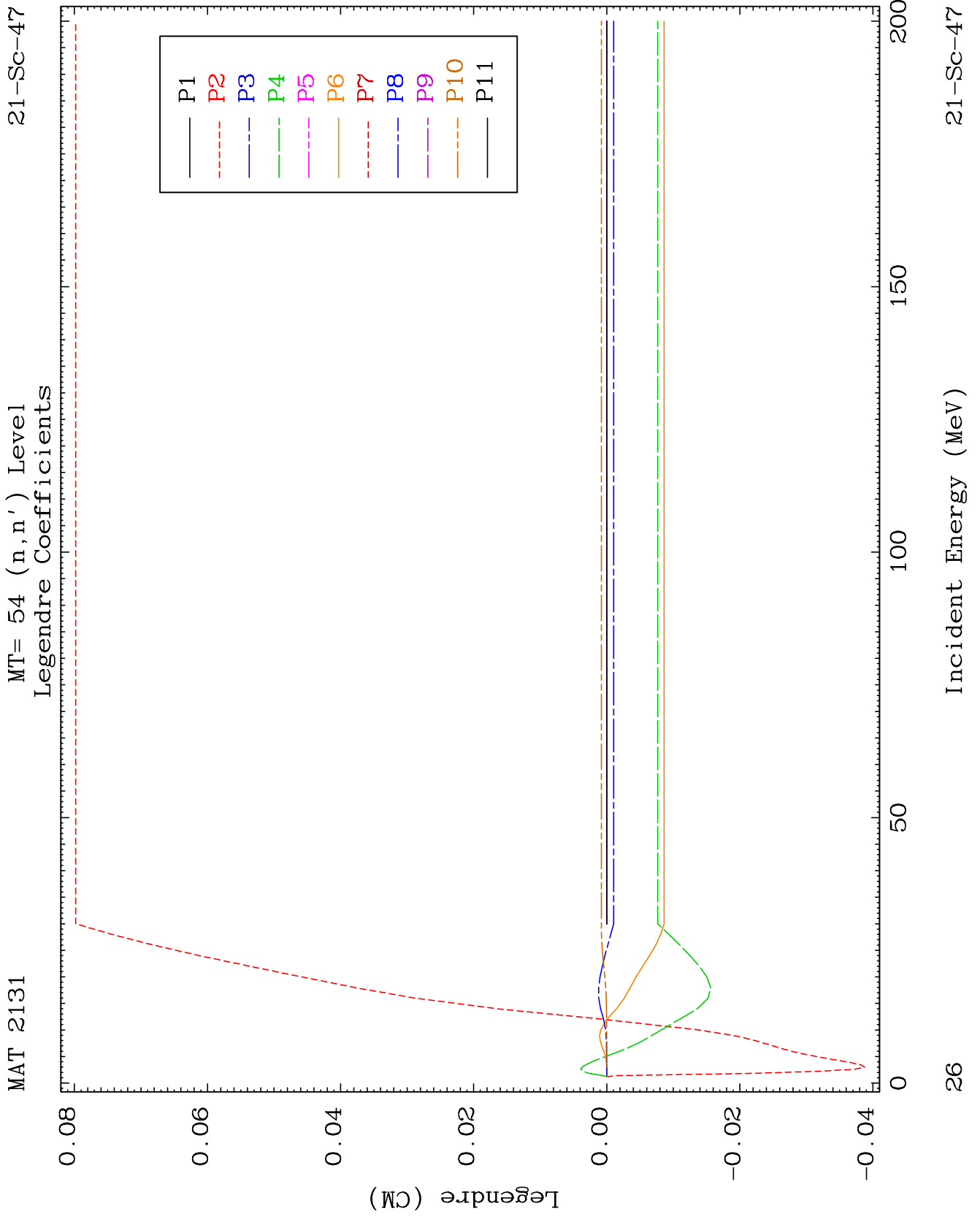








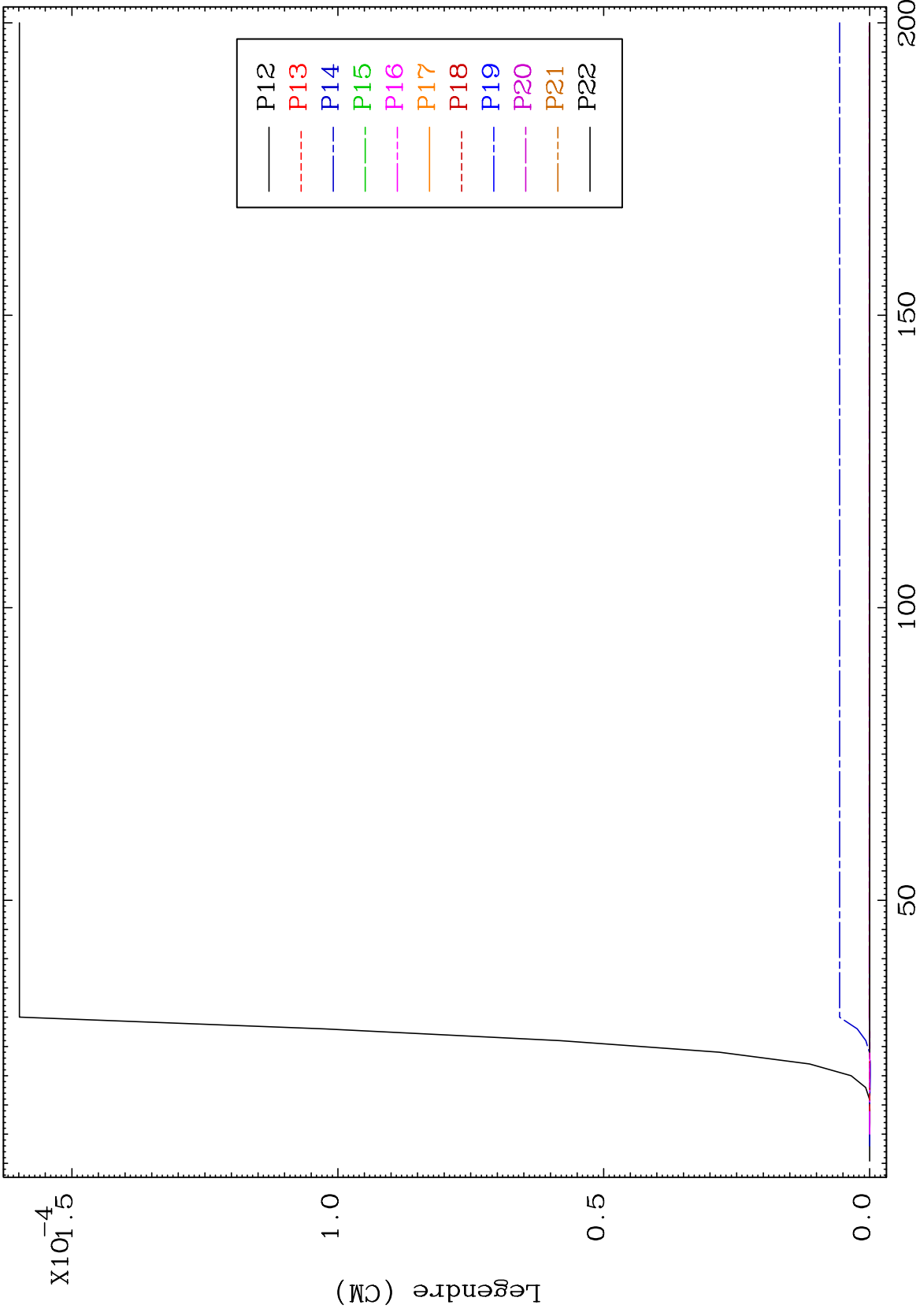




MAT 2131

MT= 54 (n,n') Level
Legendre Coefficients

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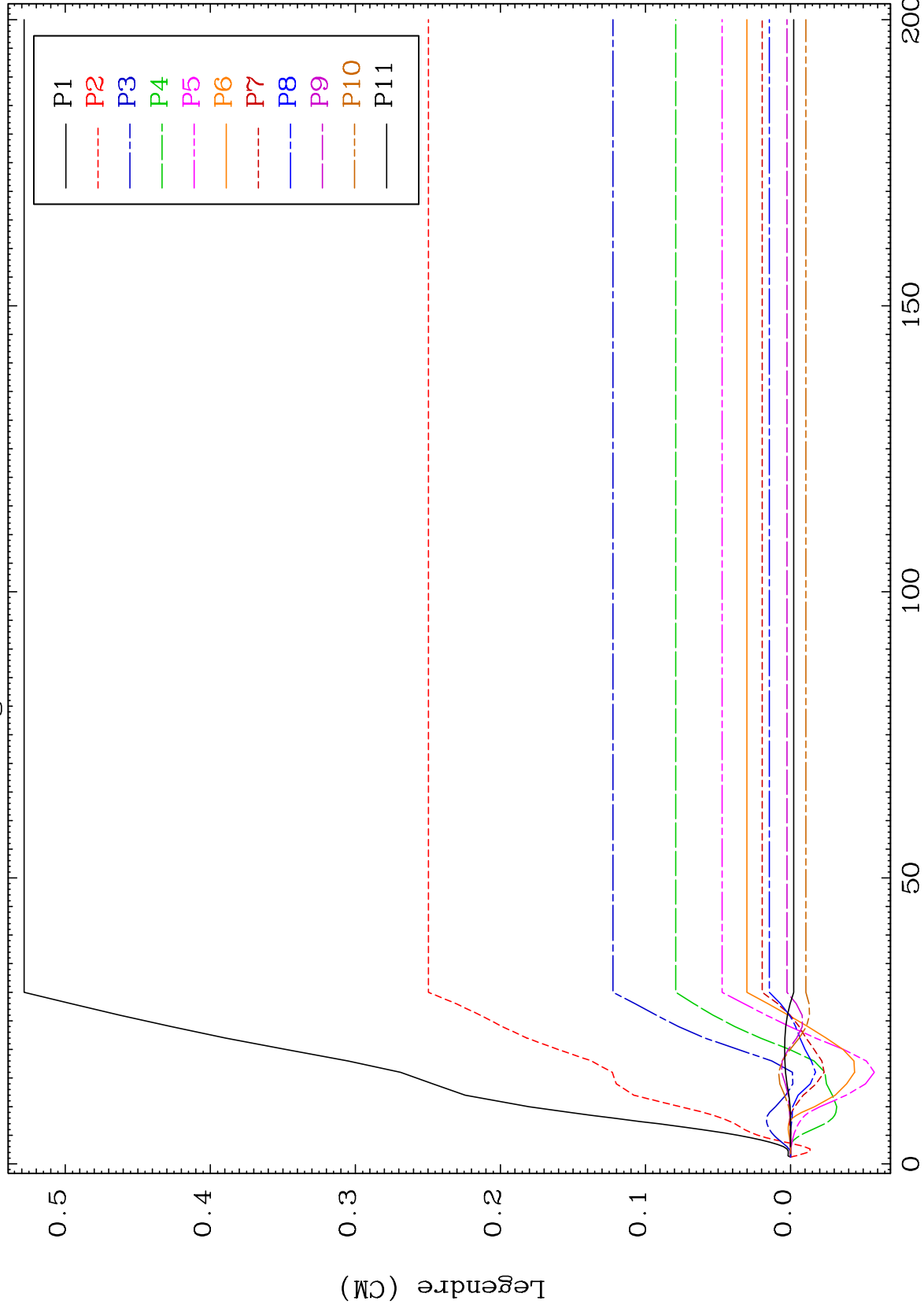
Incident Energy (MeV)

21-Sc-47

MAT 2131

MT= 55 (n,n') Level
Legendre Coefficients

21-Sc-47



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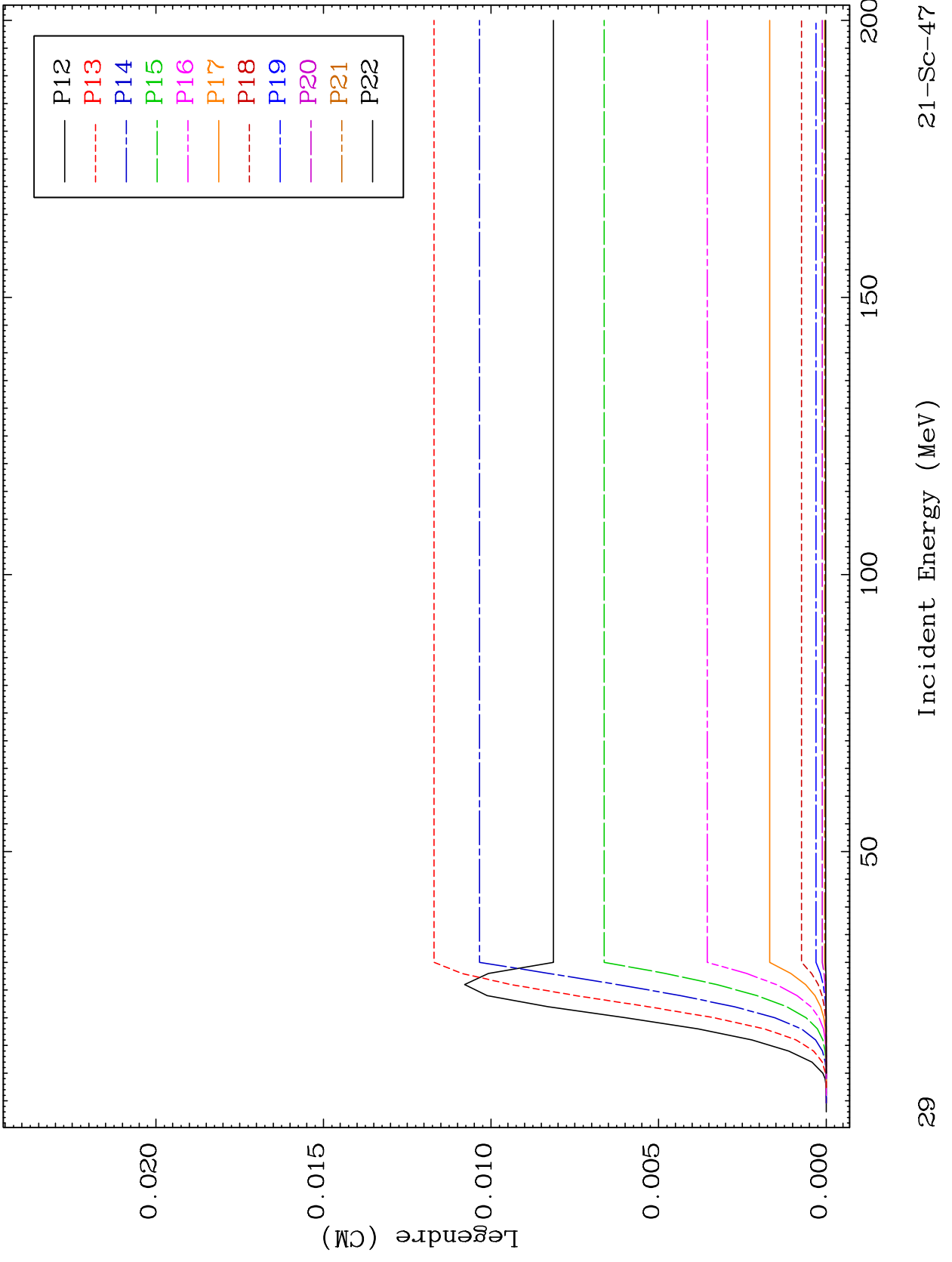
Incident Energy (MeV)

21-Sc-47

MAT 2131

MT= 55 (n,n') Level
Legendre Coefficients

21-Sc-47



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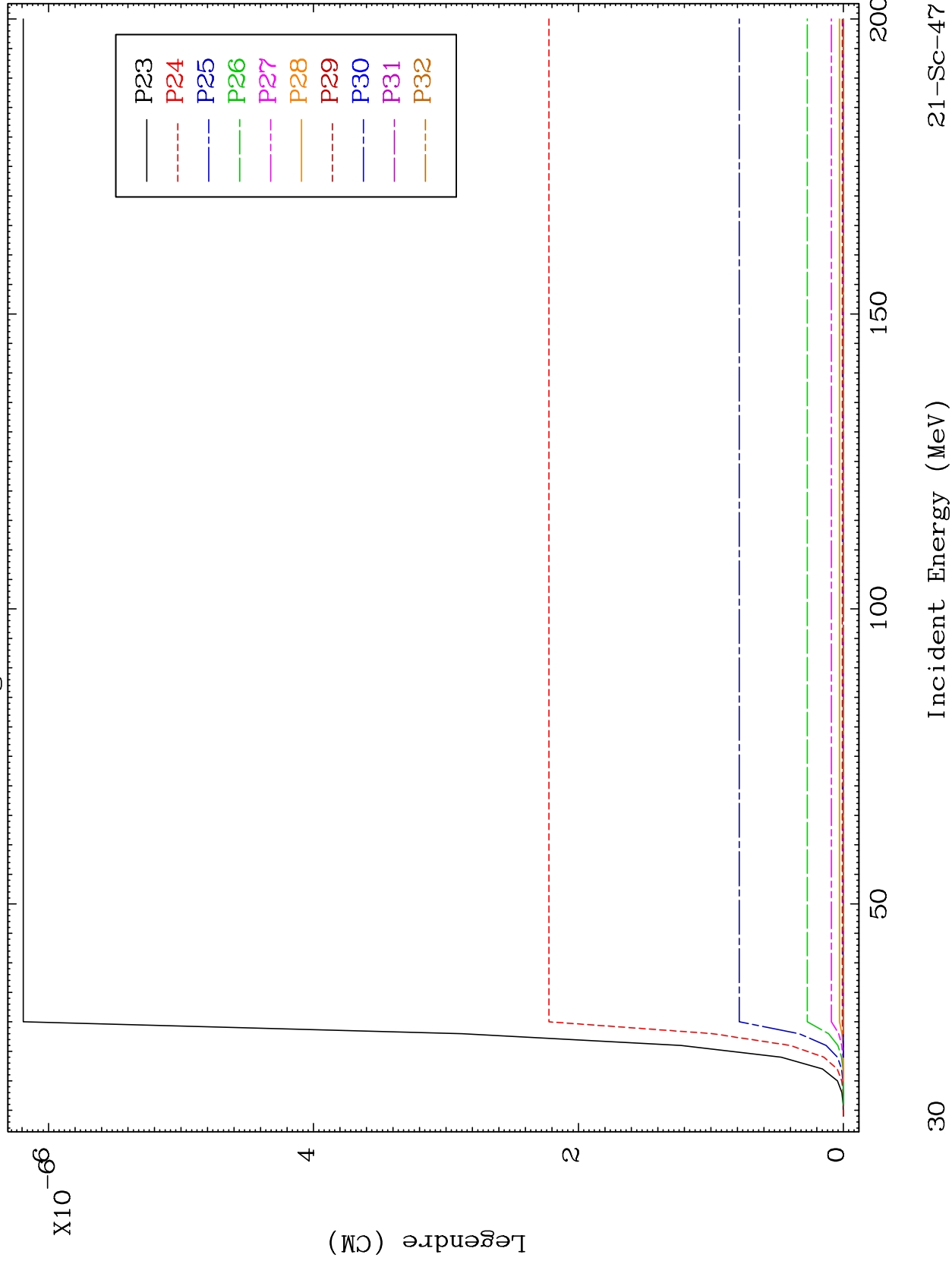
Incident Energy (MeV)

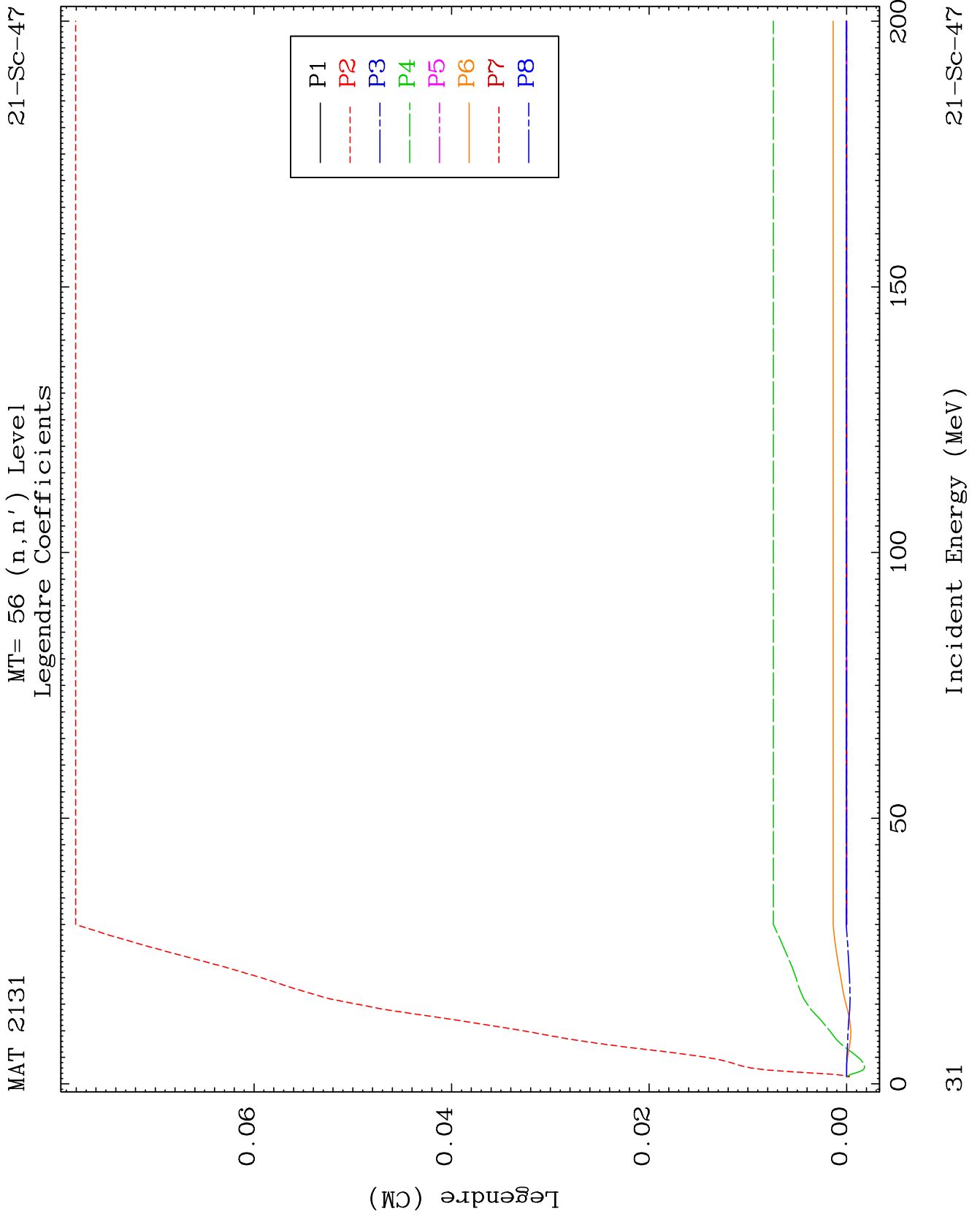
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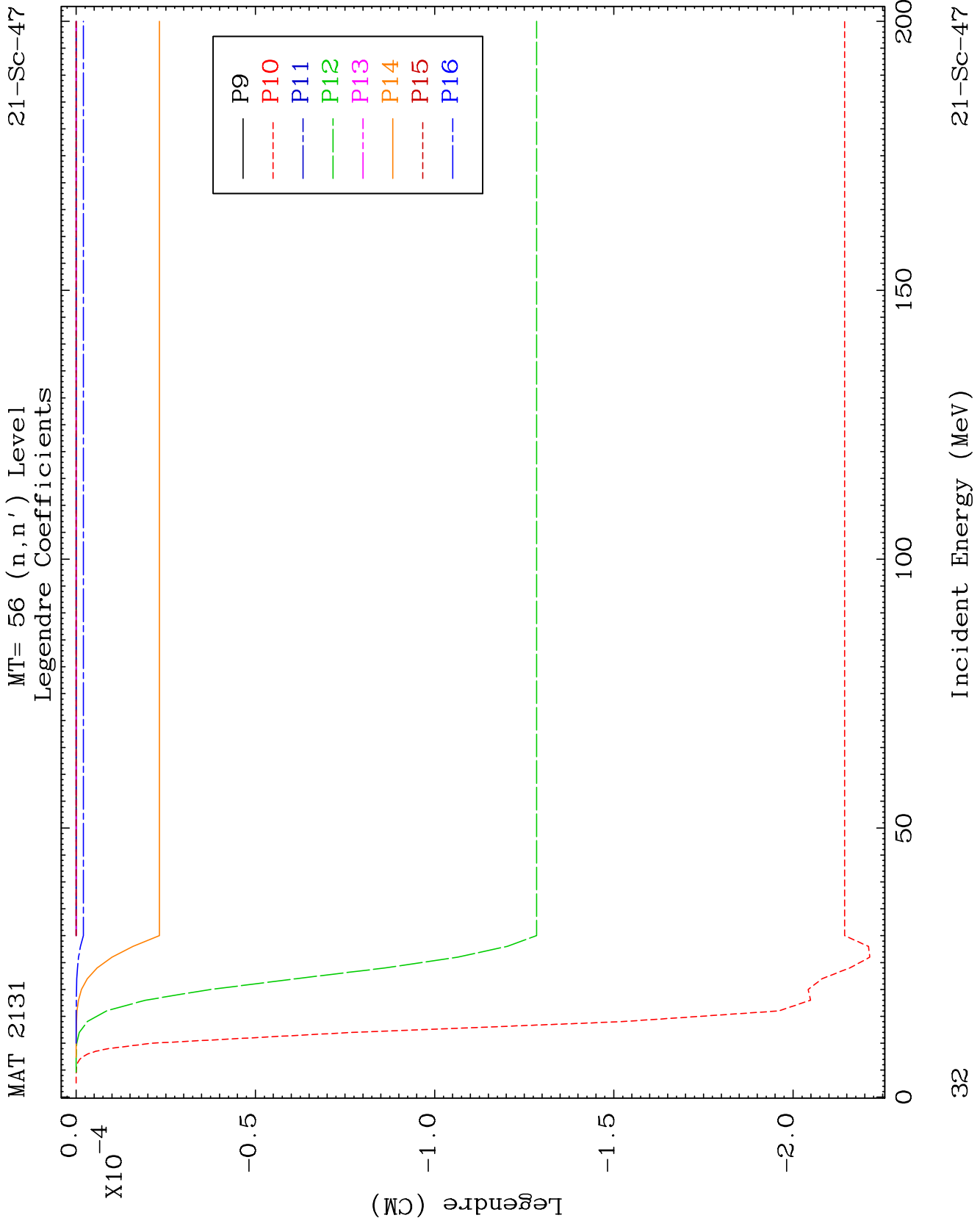
MAT 2131

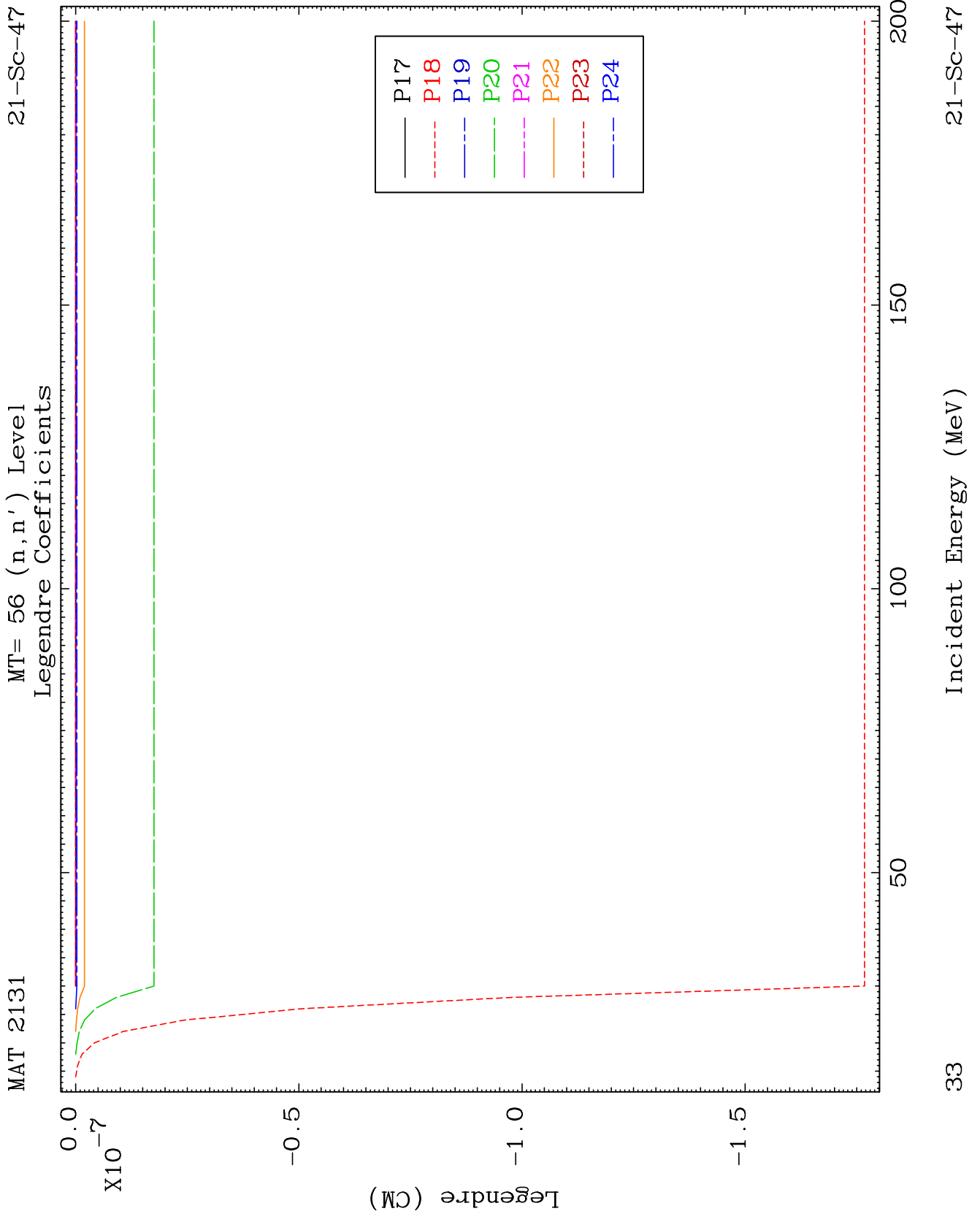
MT= 55 (n,n') Level
Legendre Coefficients

21-Sc-47





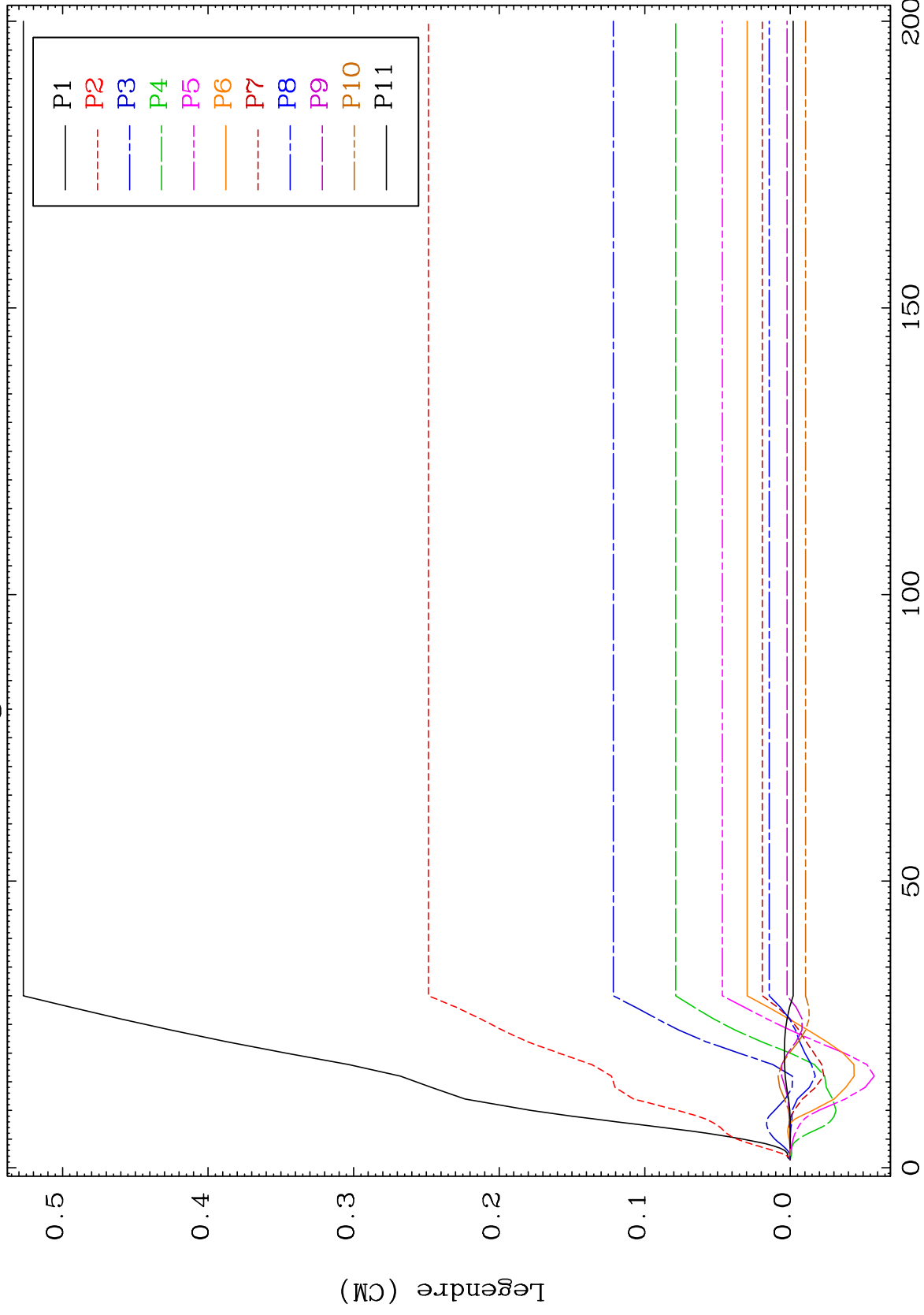




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MT= 57 (n,n') Level
Legendre Coefficients

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34

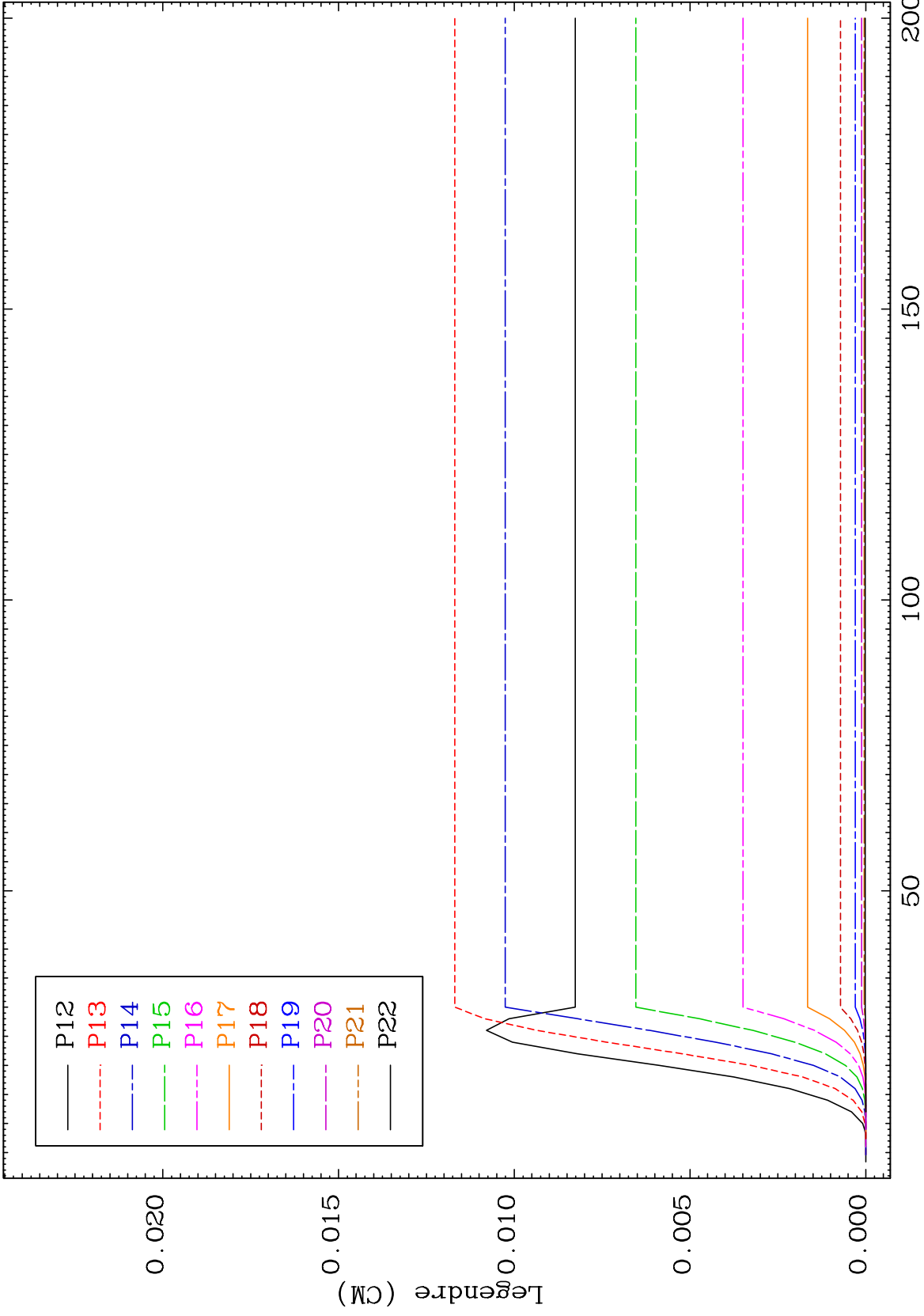
Incident Energy (MeV)

21-Sc-47

MAT 2131

MT= 57 (n,n') Level
Legendre Coefficients

21-Sc-47



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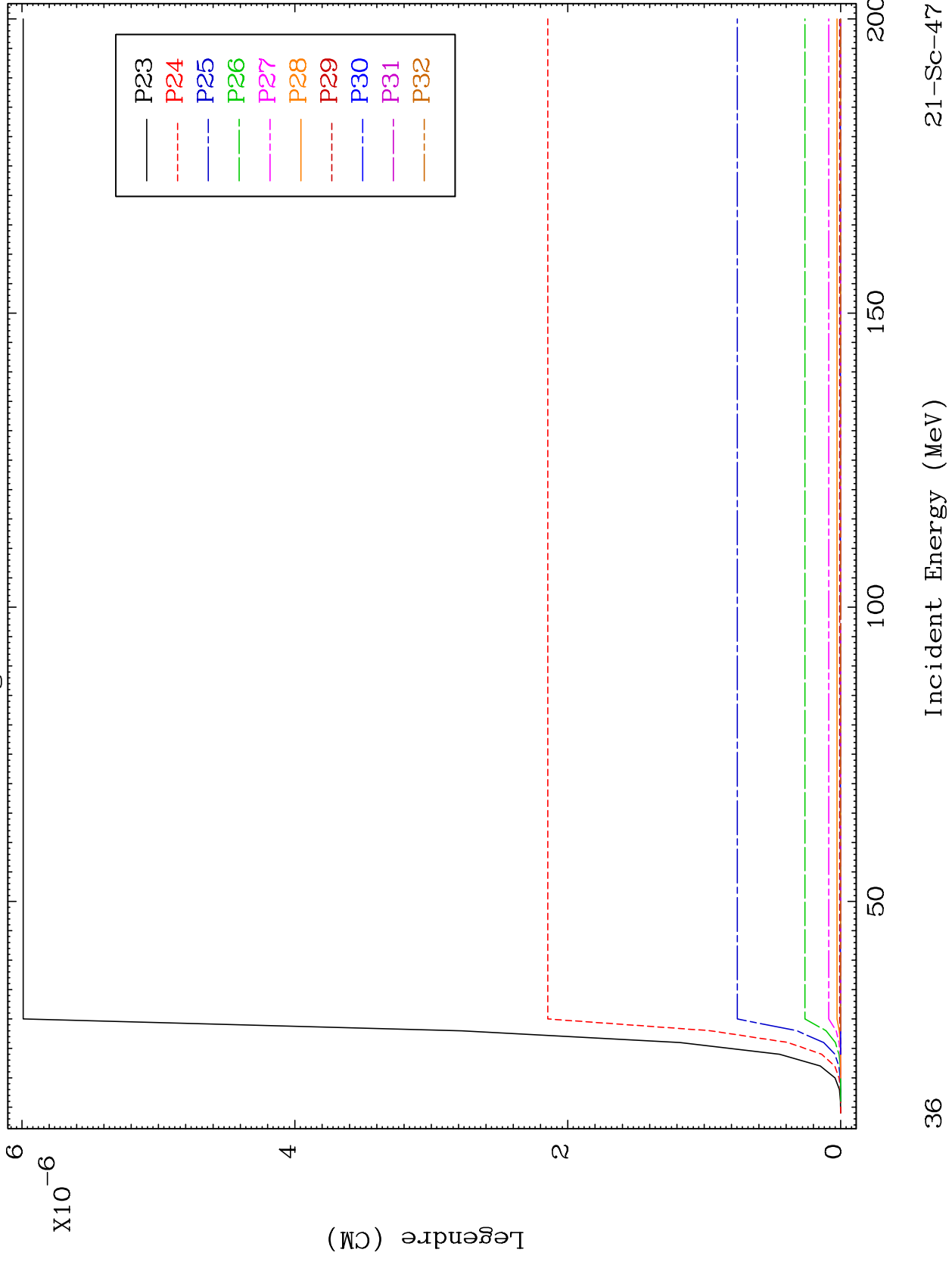
Incident Energy (MeV)

21-Sc-47

MAT 2131

MT= 57 (n,n') Level
Legendre Coefficients

21-Sc-47



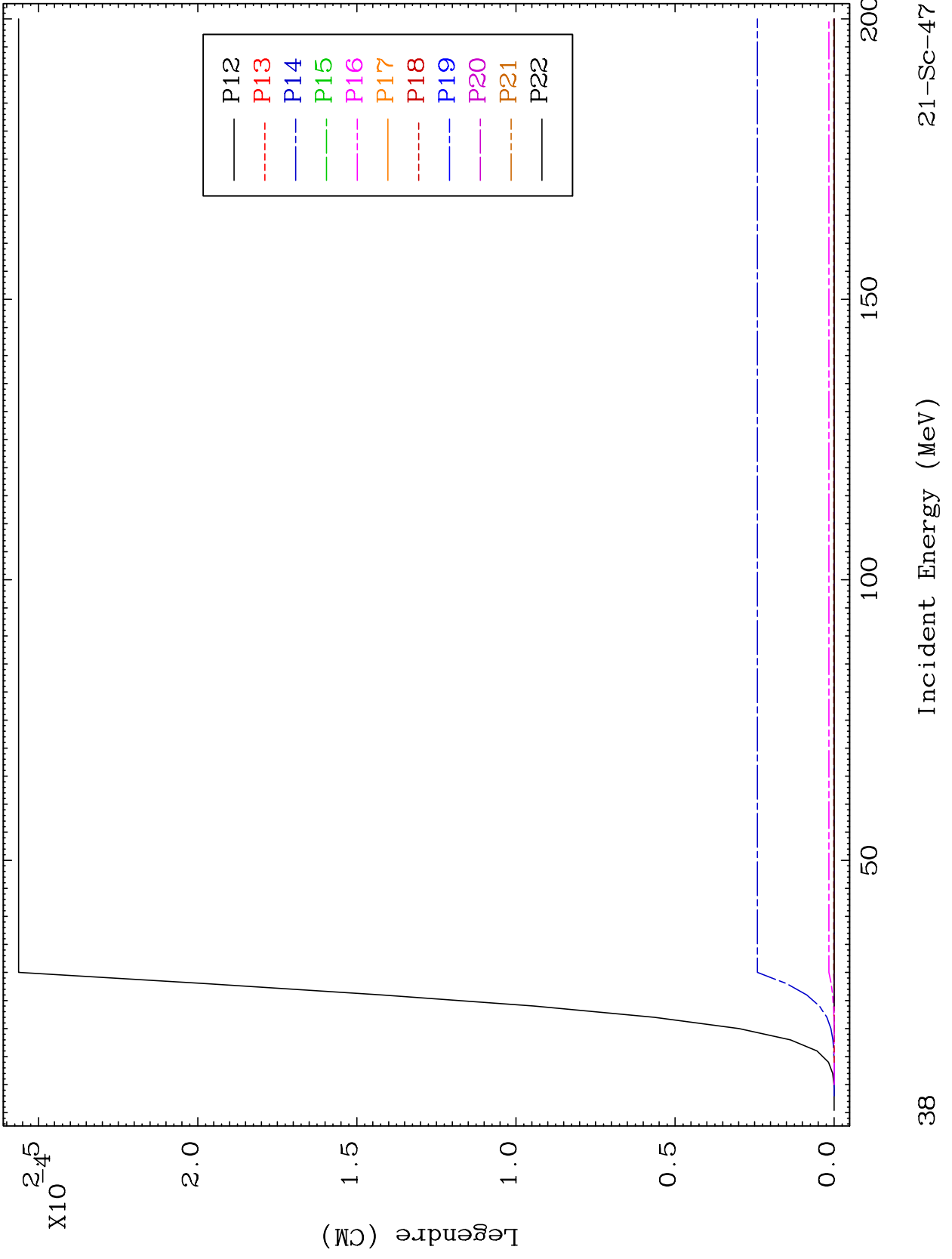
36

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MAT 2131

MT= 58 (n,n') Level
Legendre Coefficients

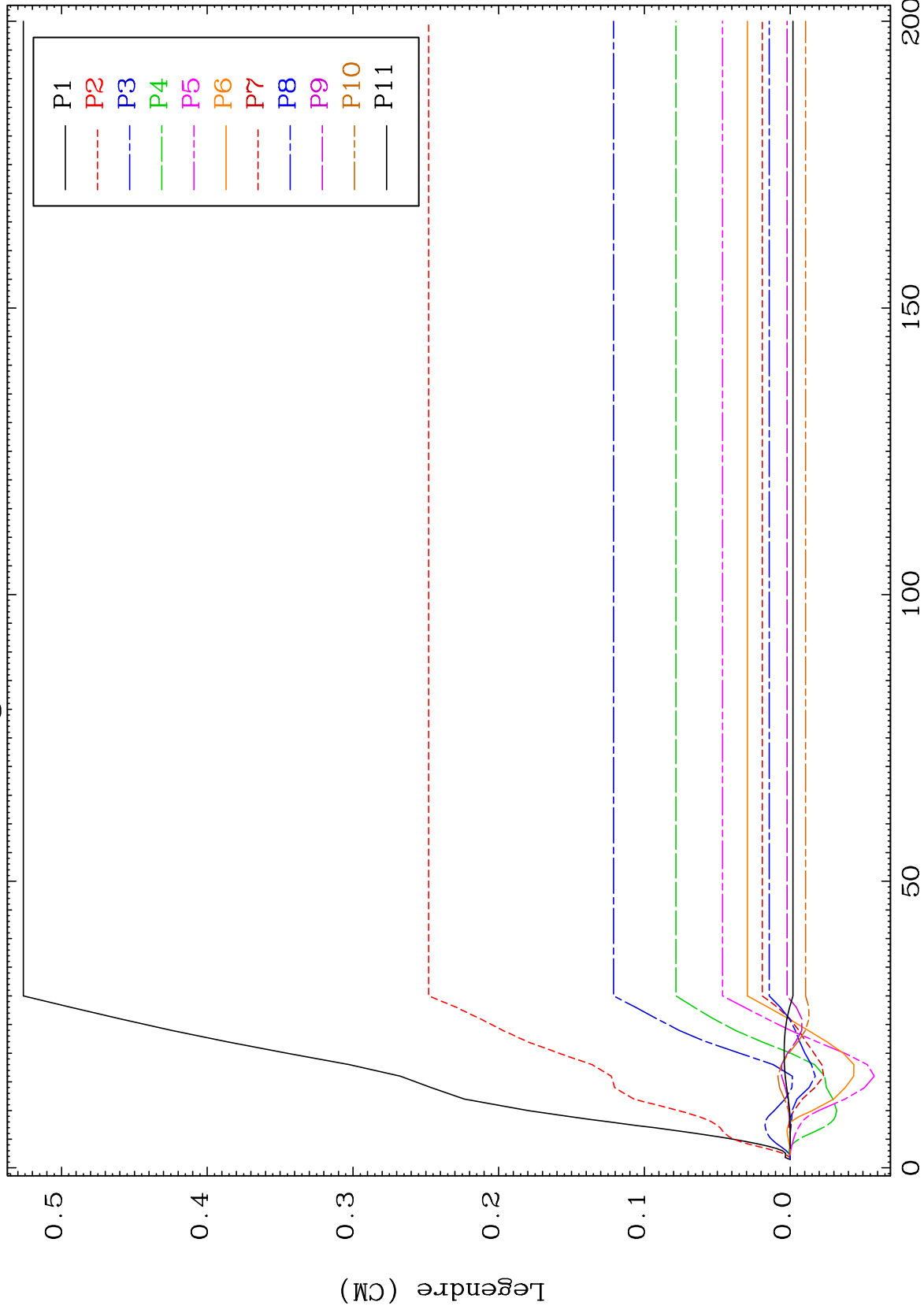
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MAT 2131

MT= 59 (n,n') Level
Legendre Coefficients

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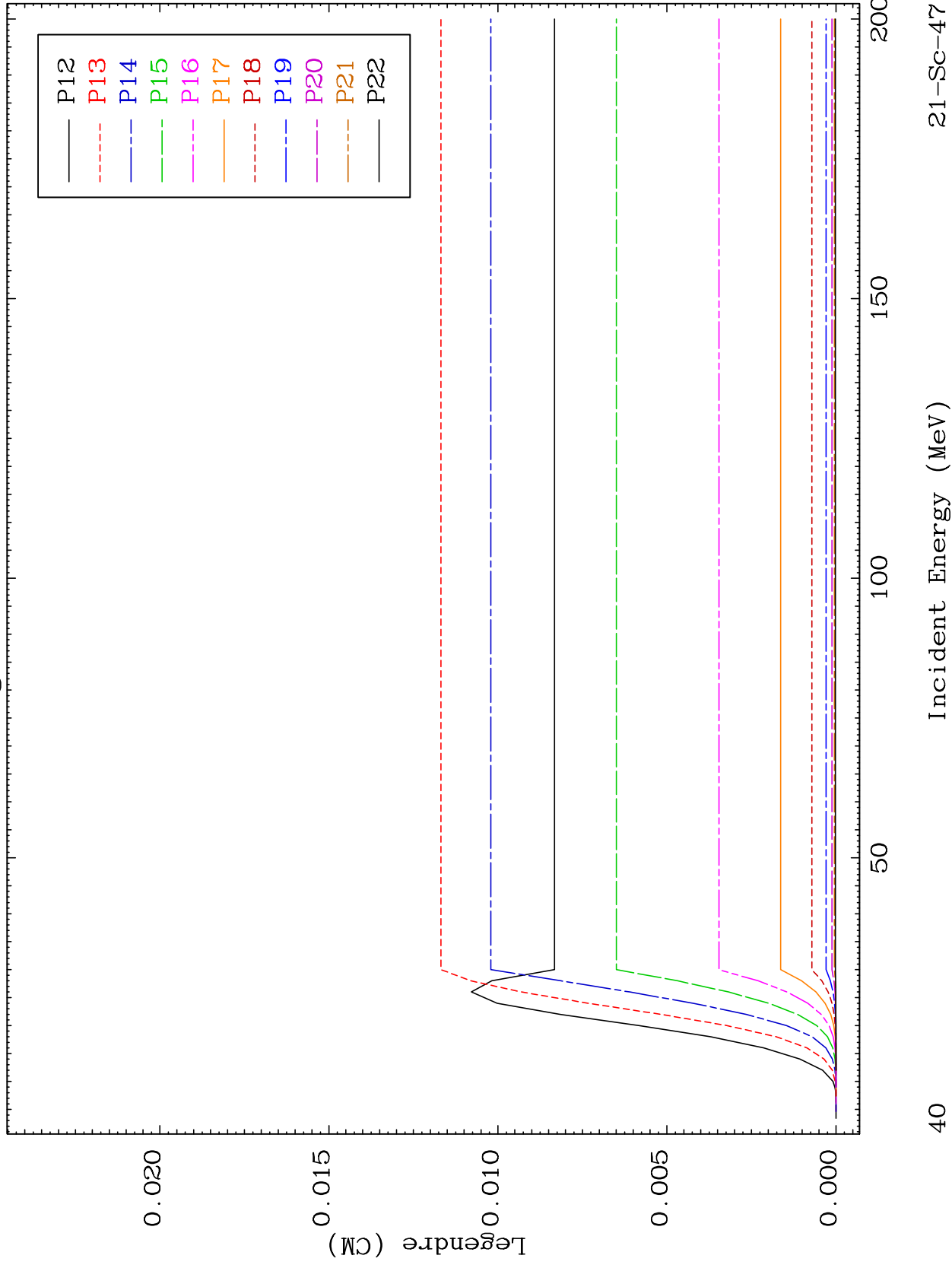
Incident Energy (MeV)

21-Sc-47

MAT 2131

MT= 59 (n,n') Level
Legendre Coefficients

21-Sc-47



21-Sc-47

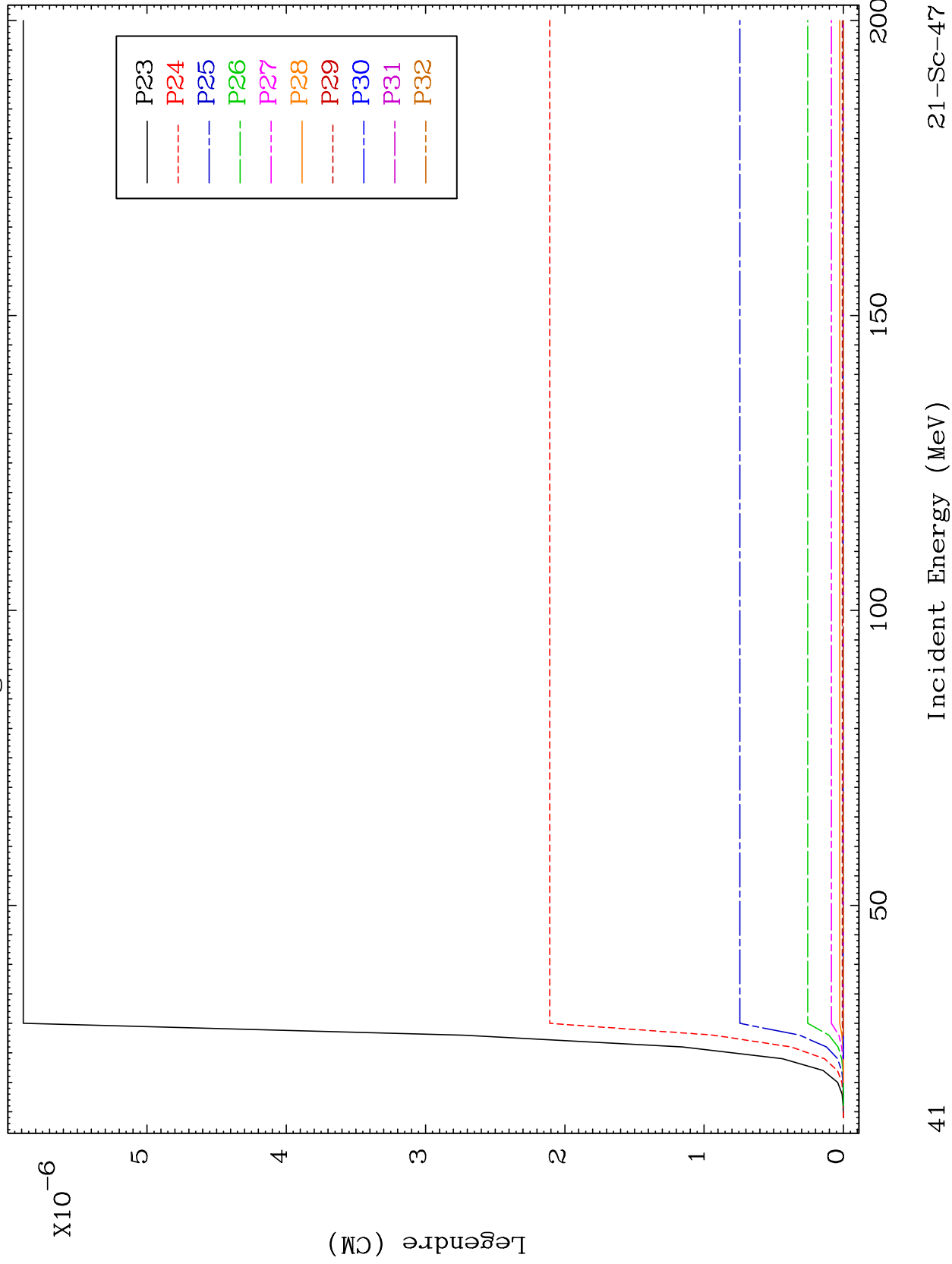
Incident Energy (MeV)

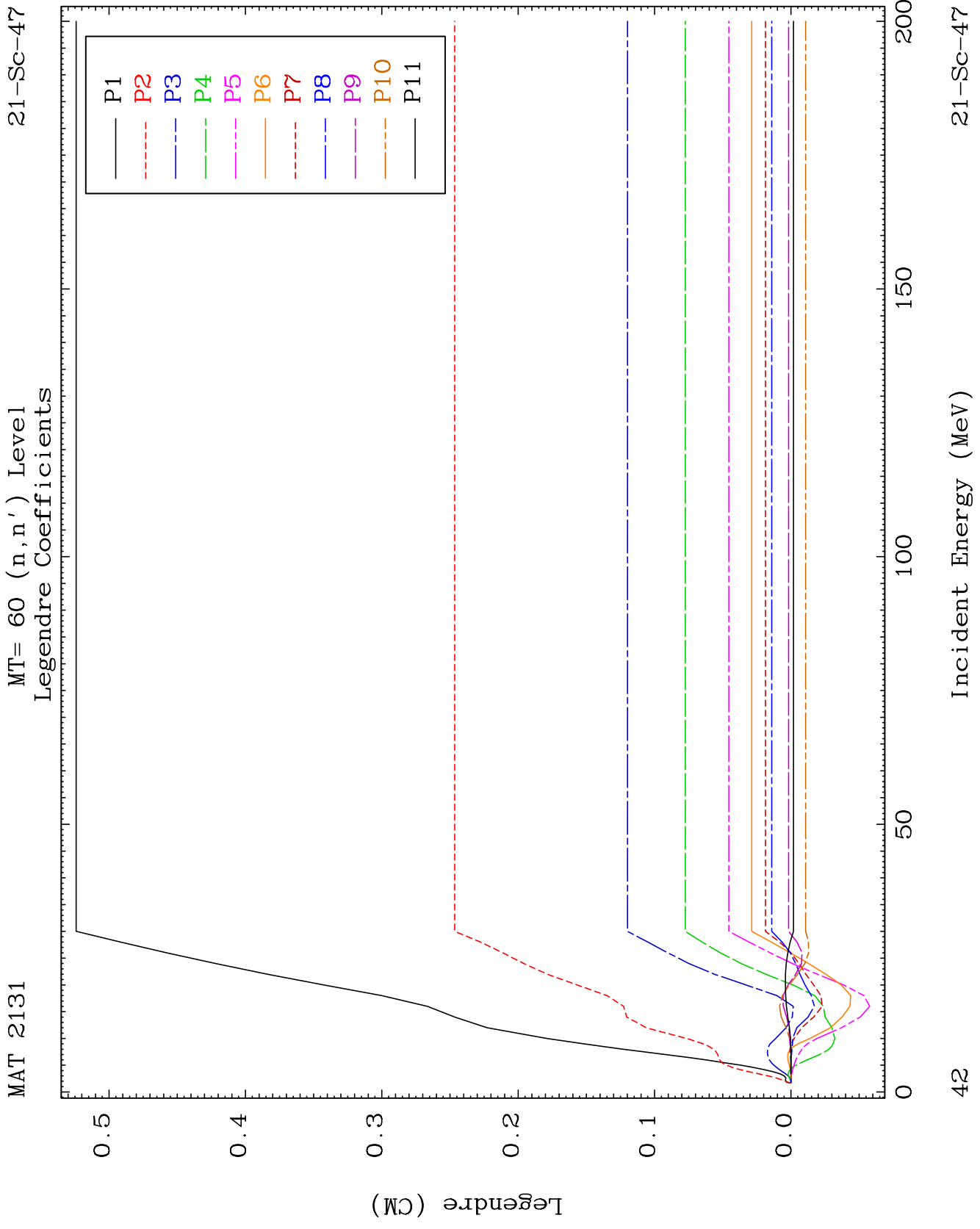
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MAT 2131

MT= 59 (n,n') Level
Legendre Coefficients

21-Sc-47

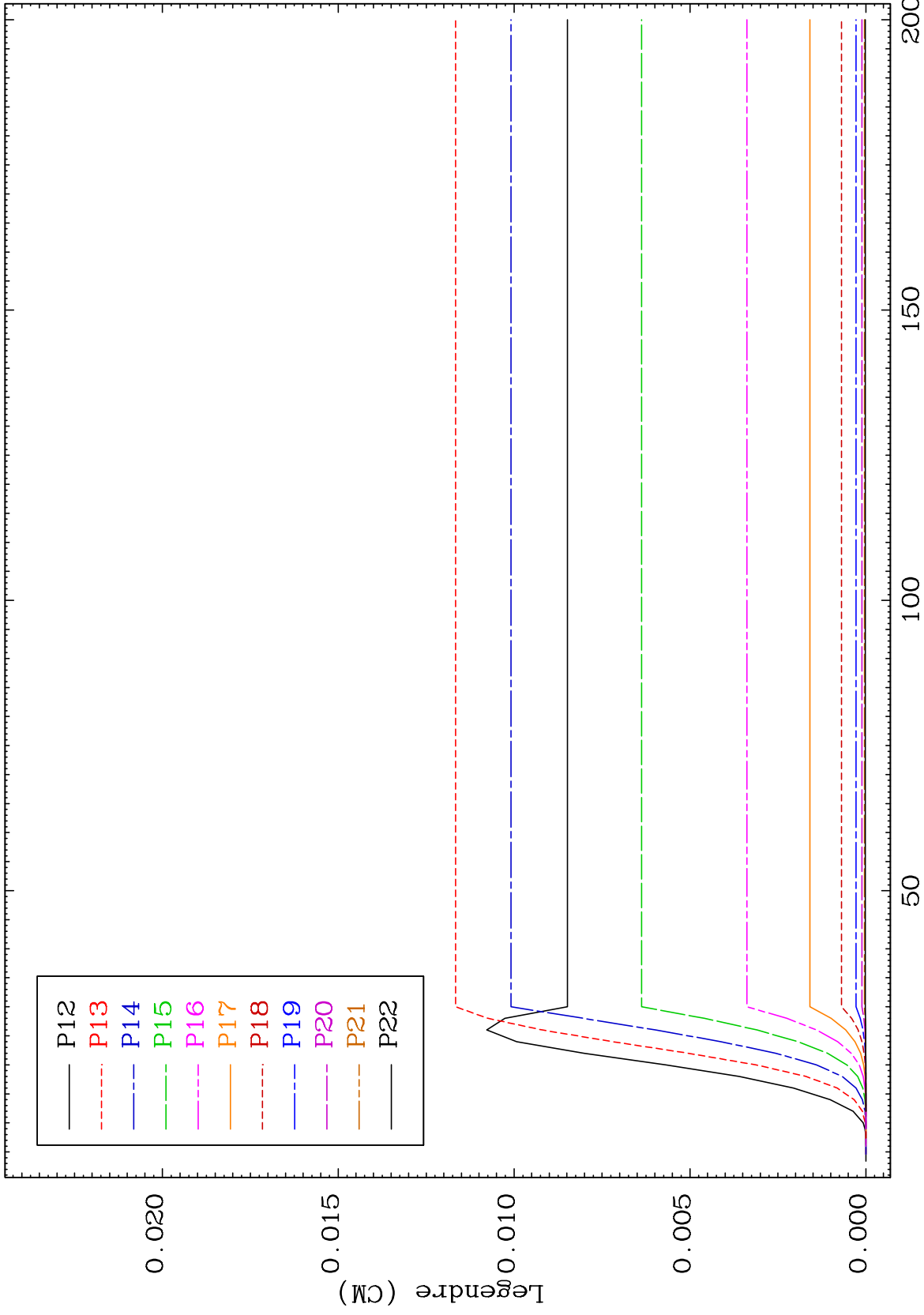




MAT 2131

MT= 60 (n,n') Level
Legendre Coefficients

21-Sc-47



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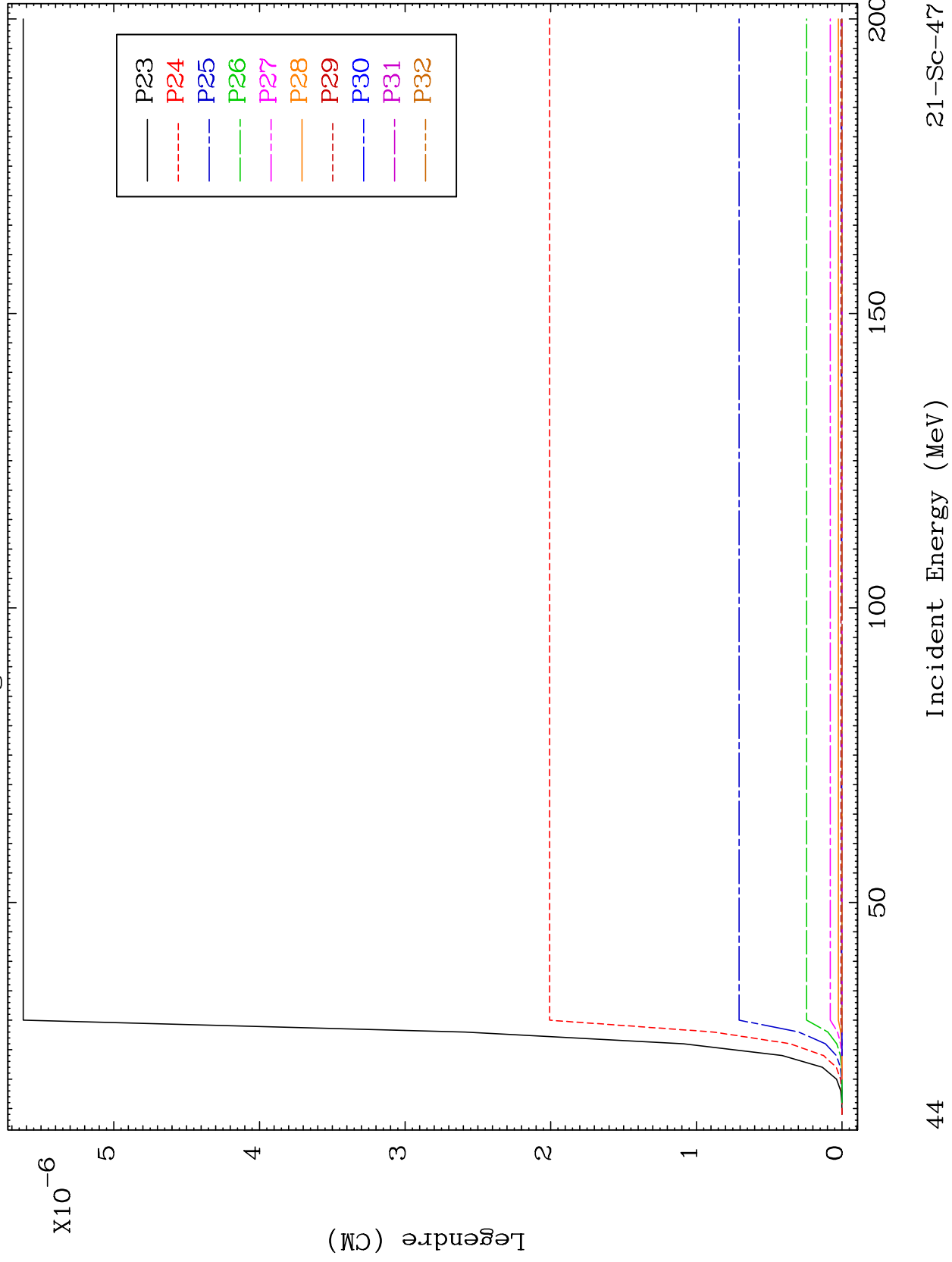
Incident Energy (MeV)

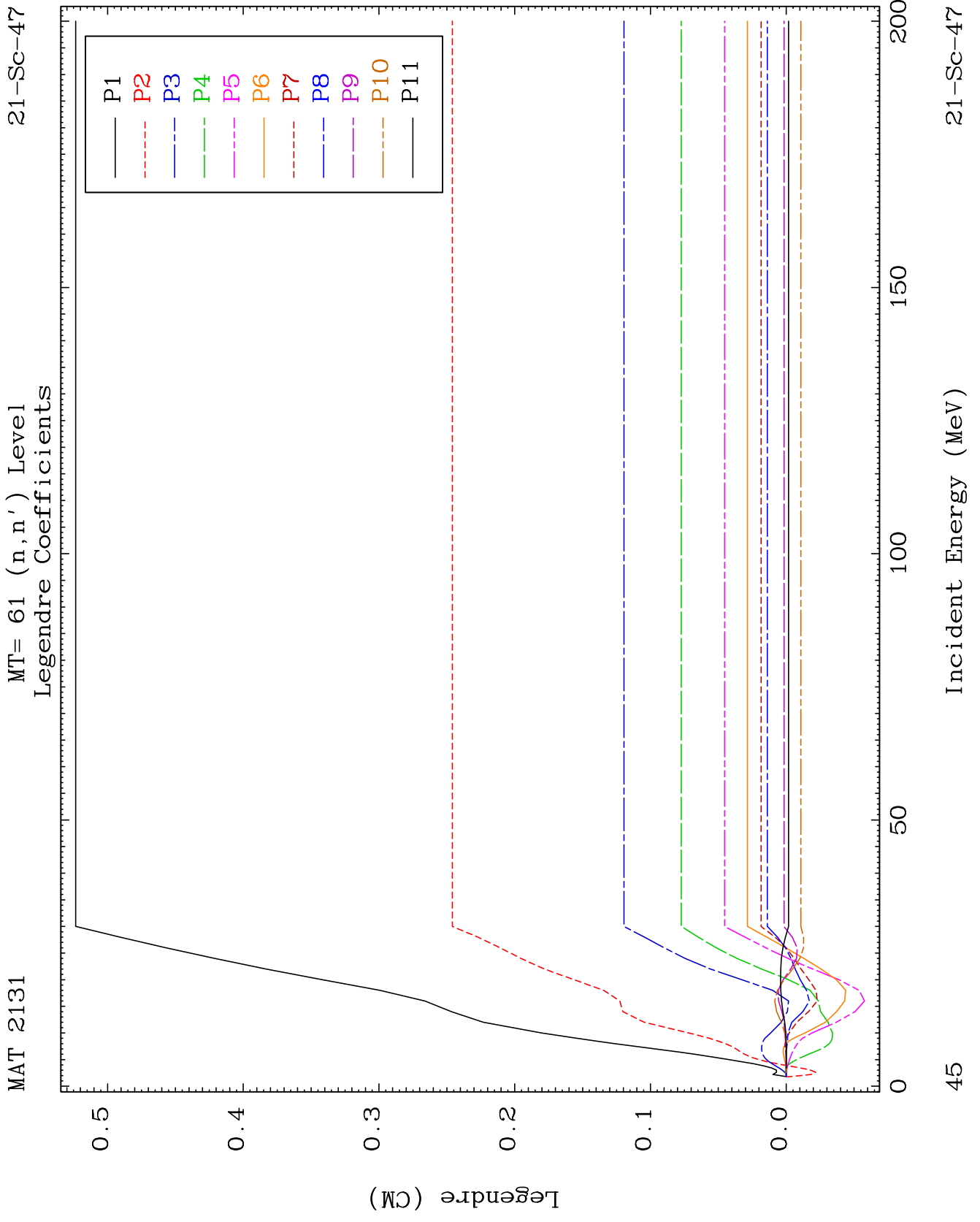
21-Sc-47

MAT 2131

MT= 60 (n,n') Level
Legendre Coefficients

21-Sc-47

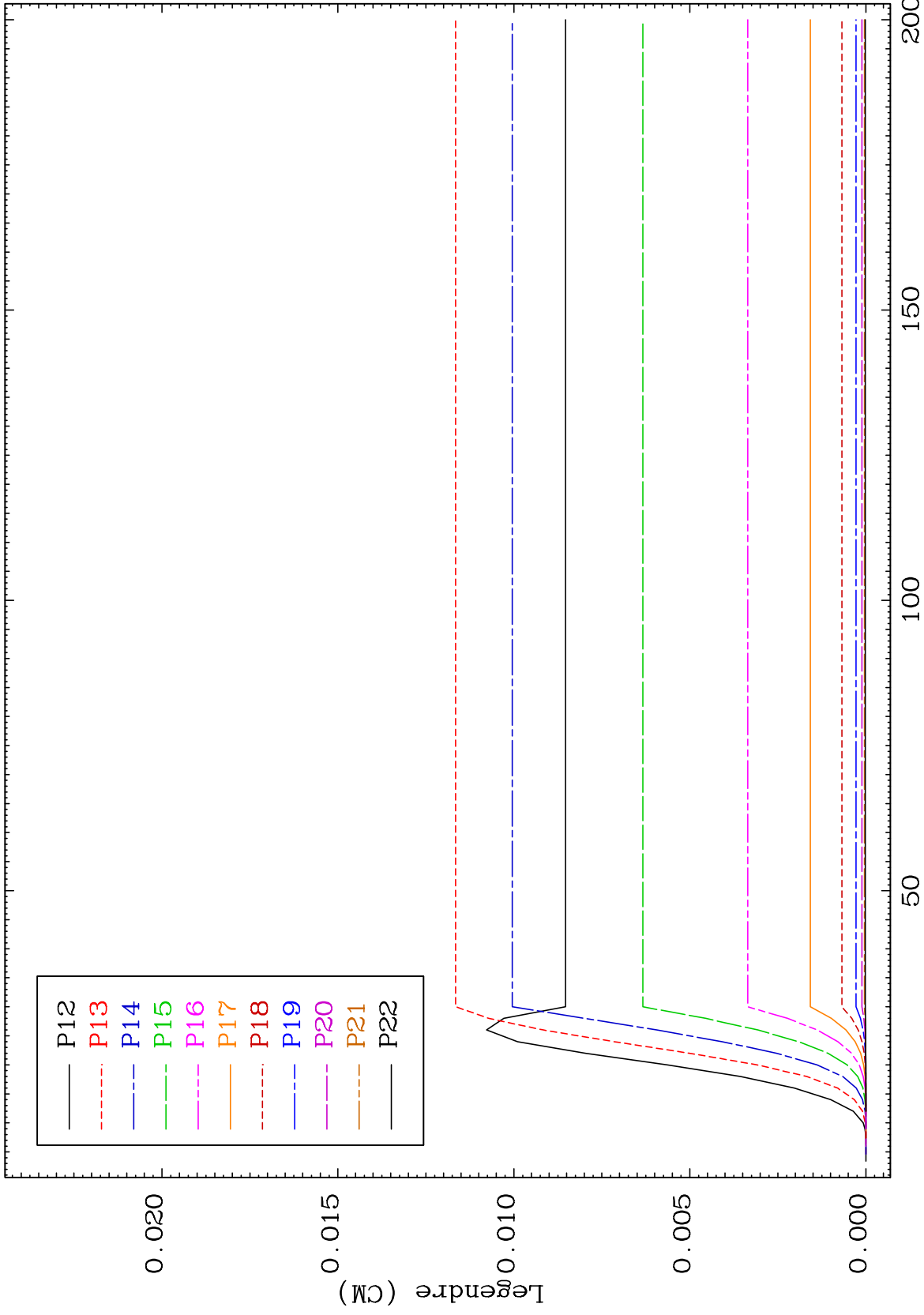




MAT 2131

MT= 61 (n,n') Level
Legendre Coefficients

21-Sc-47



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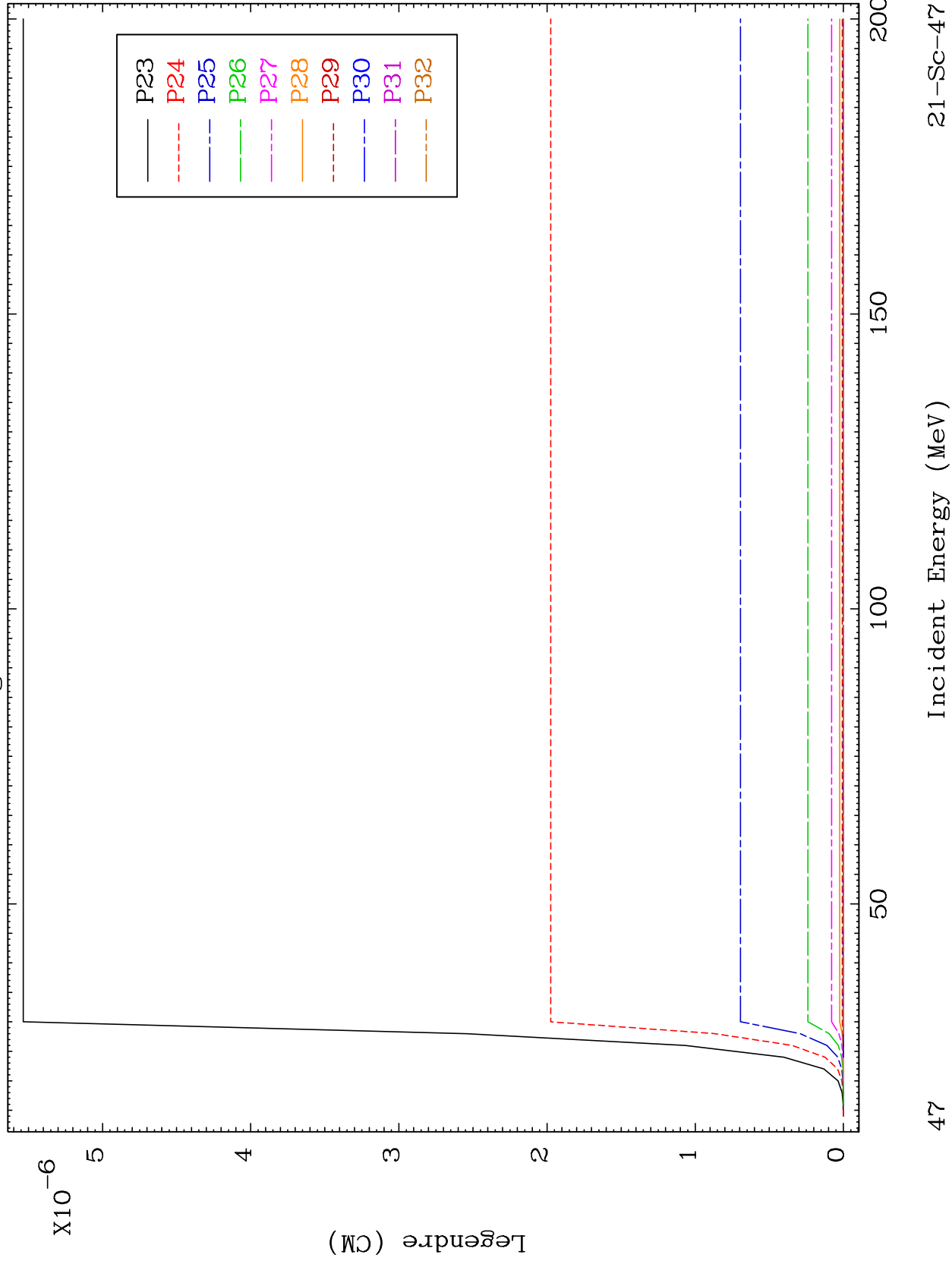
Incident Energy (MeV)

21-Sc-47

MAT 2131

MT= 61 (n,n') Level
Legendre Coefficients

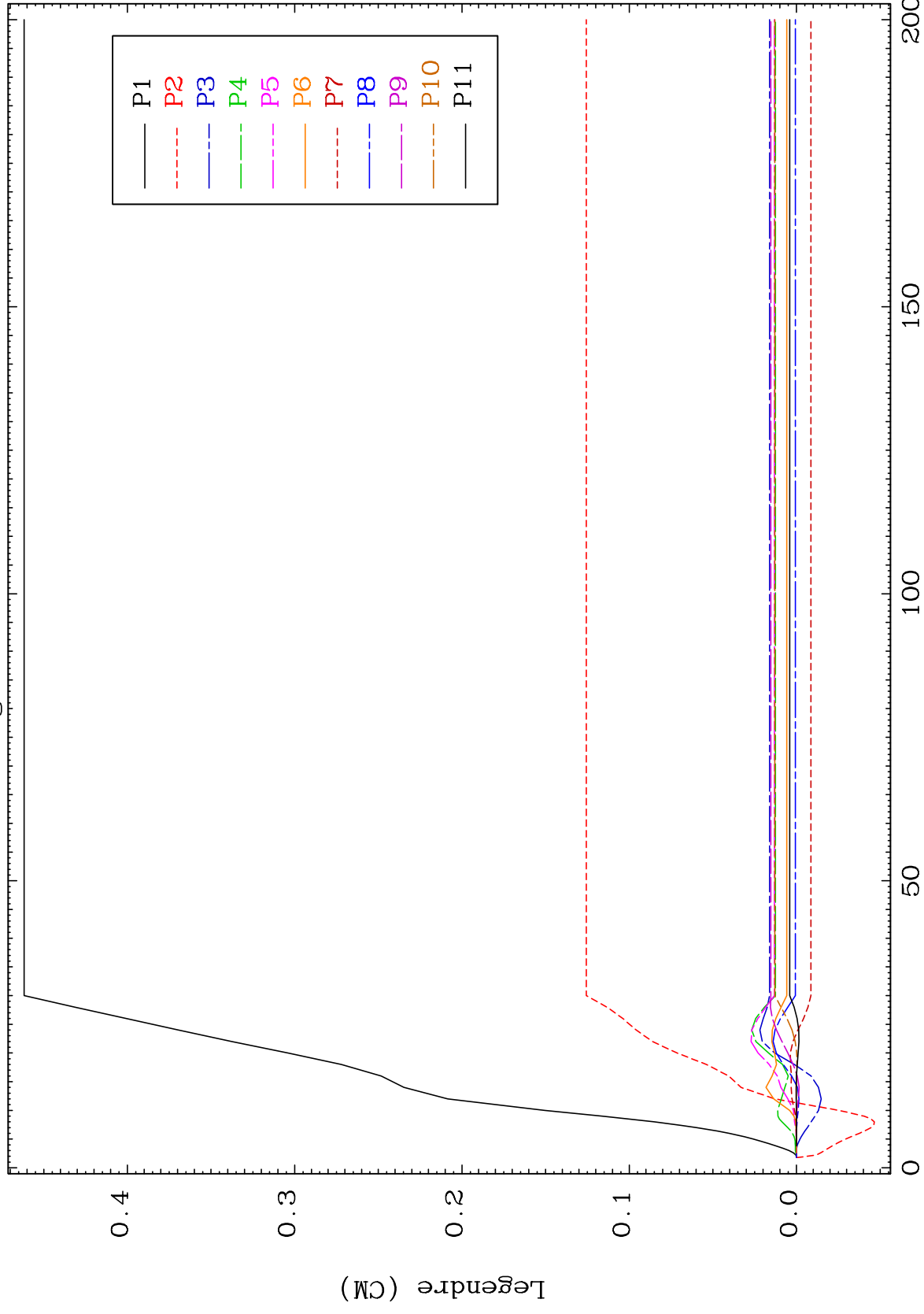
21-Sc-47



MAT 2131

MT= 62 (n,n') Level
Legendre Coefficients

21-Sc-47



48

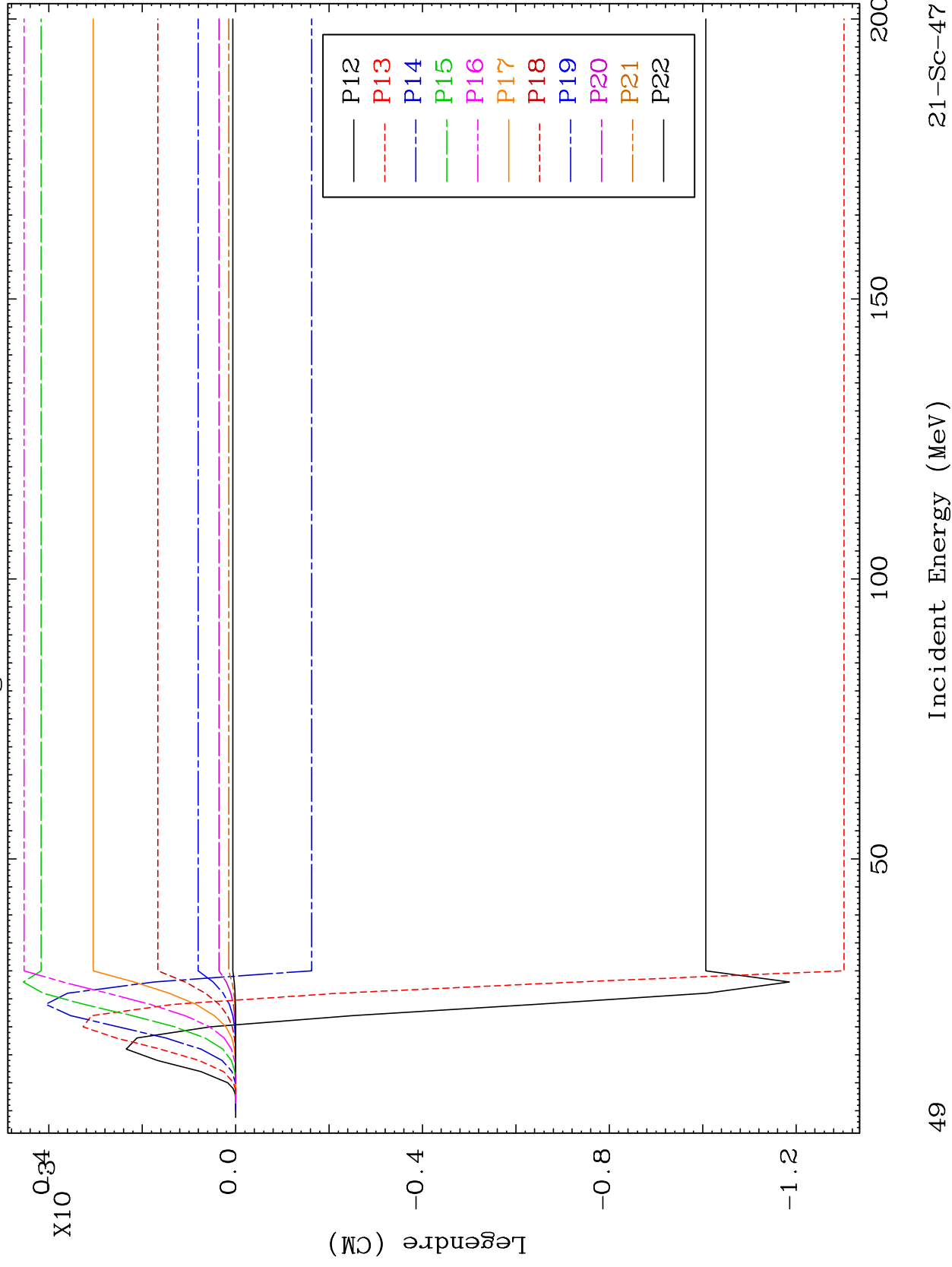
Incident Energy (MeV)

21-Sc-47

MAT 2131

21-Sc-47

MT= 62 (n,n') Level
Legendre Coefficients



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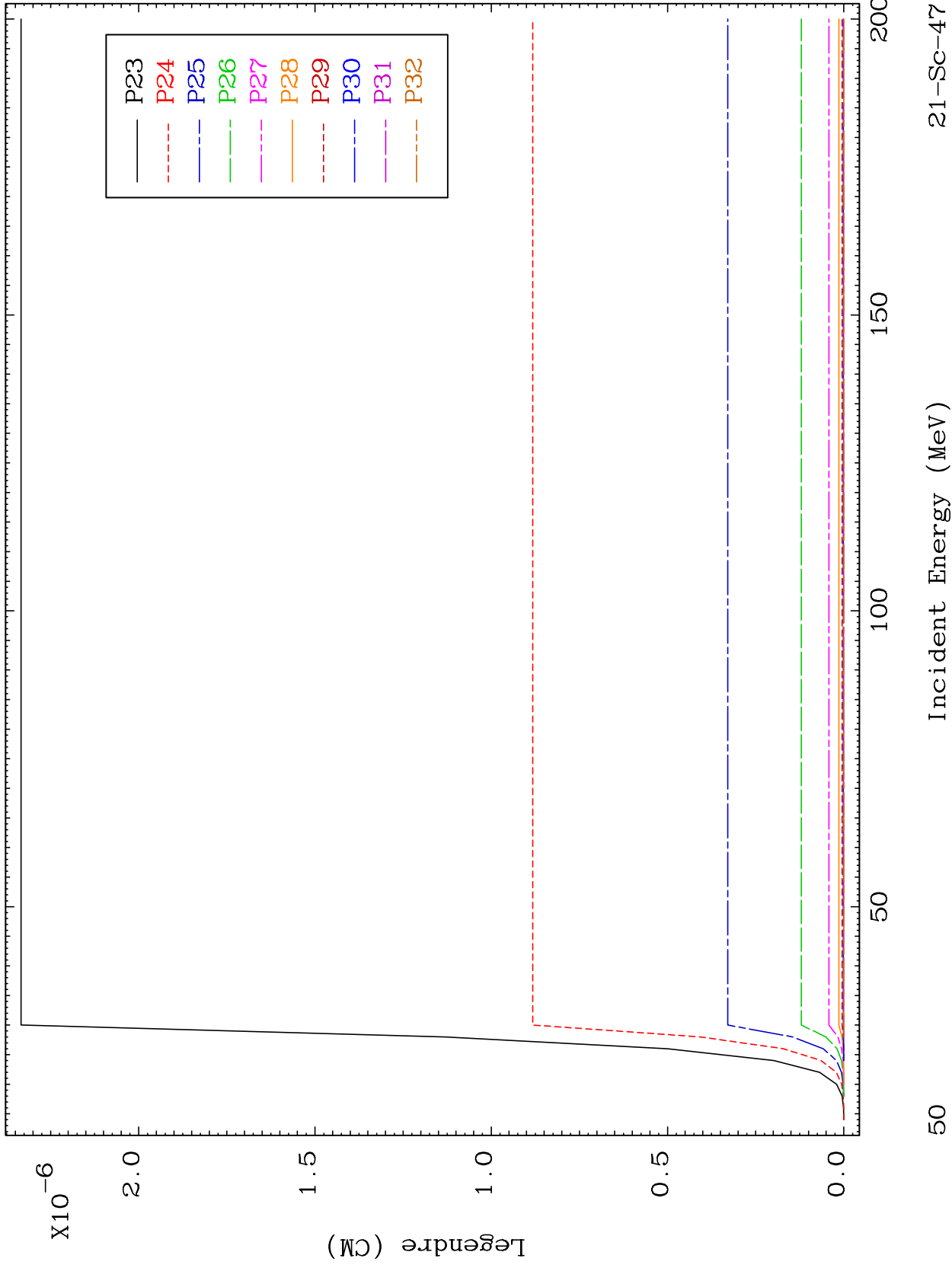
Incident Energy (MeV)

21-Sc-47

MAT 2131

MT= 62 (n,n') Level
Legendre Coefficients

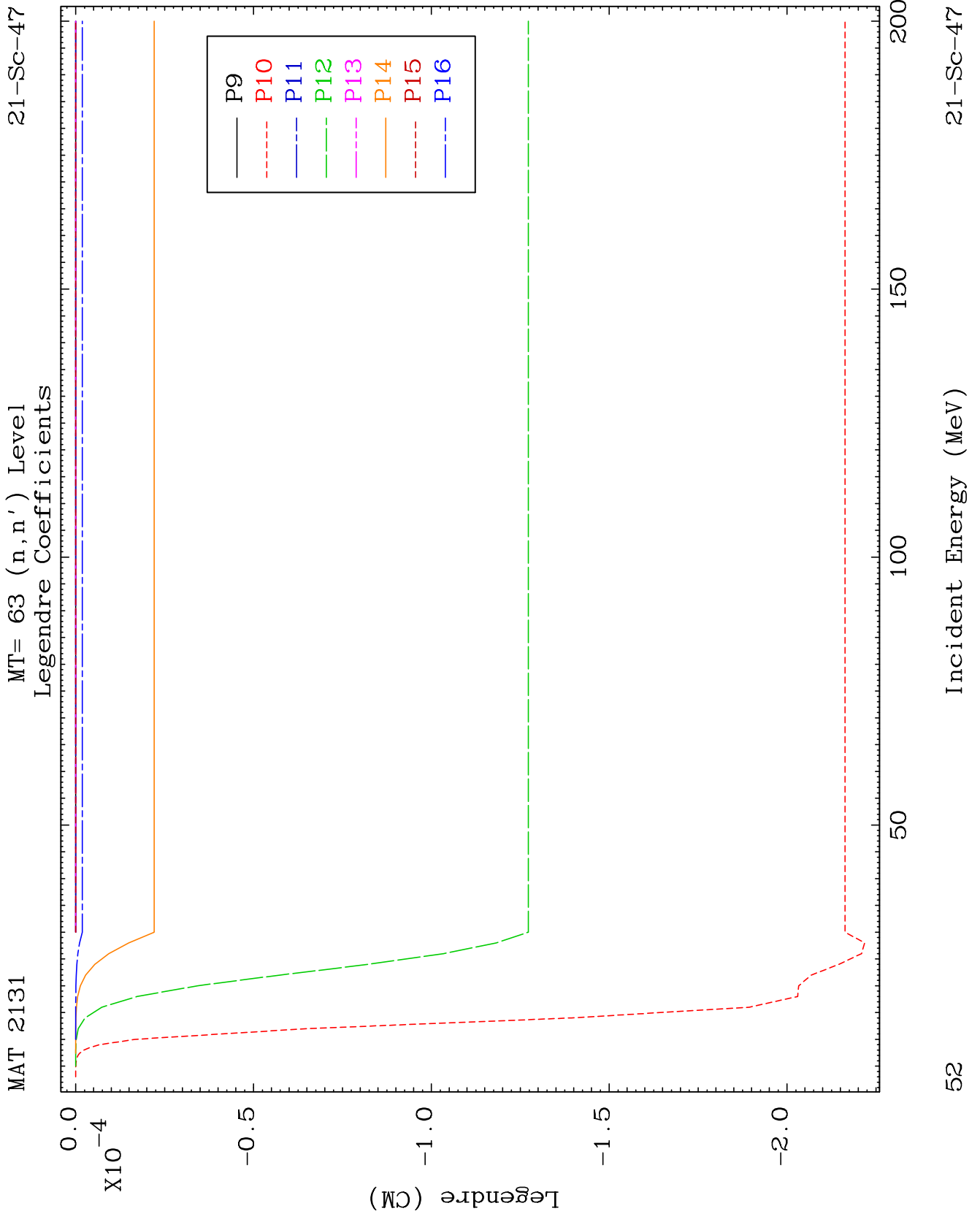
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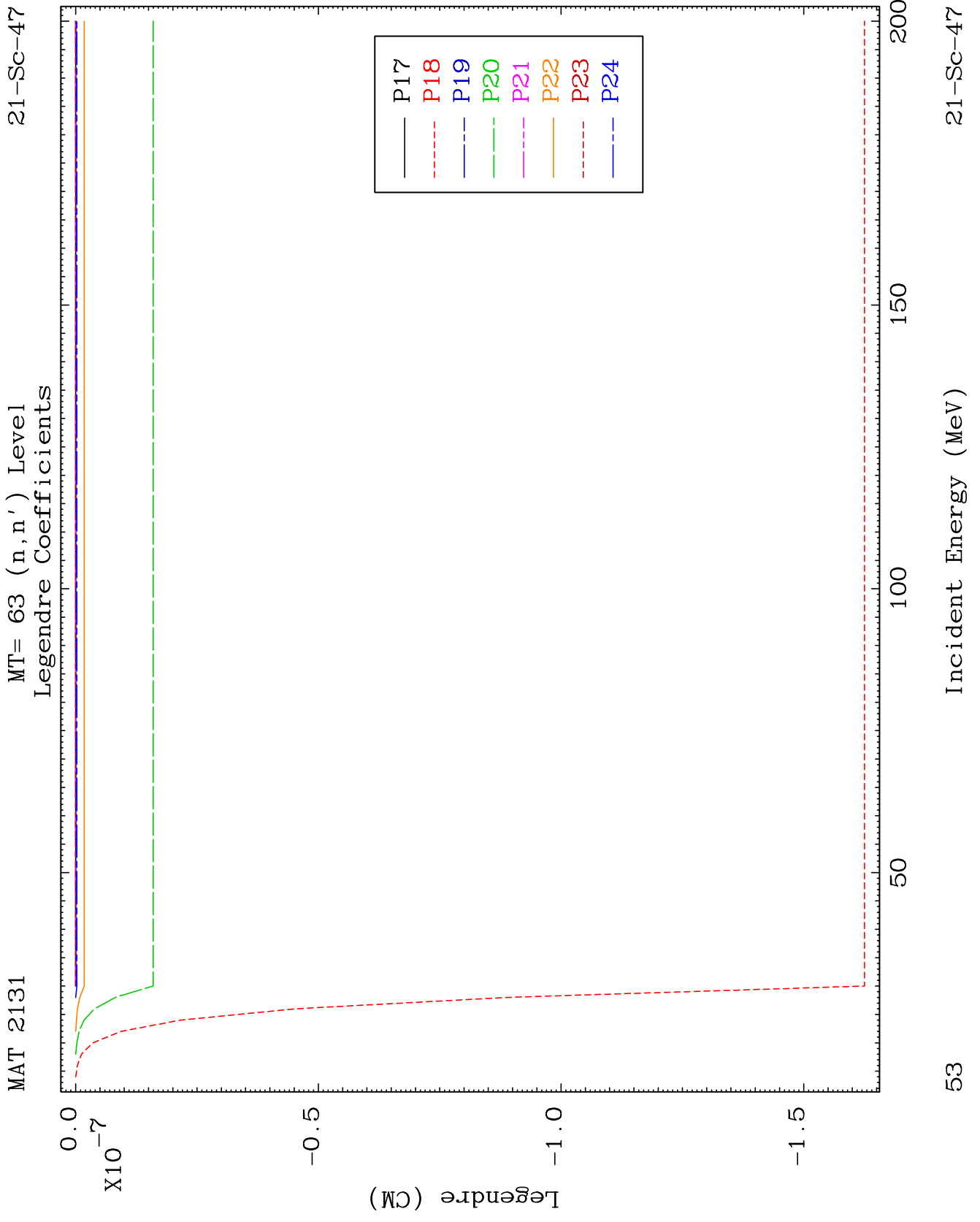


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Incident Energy (MeV)

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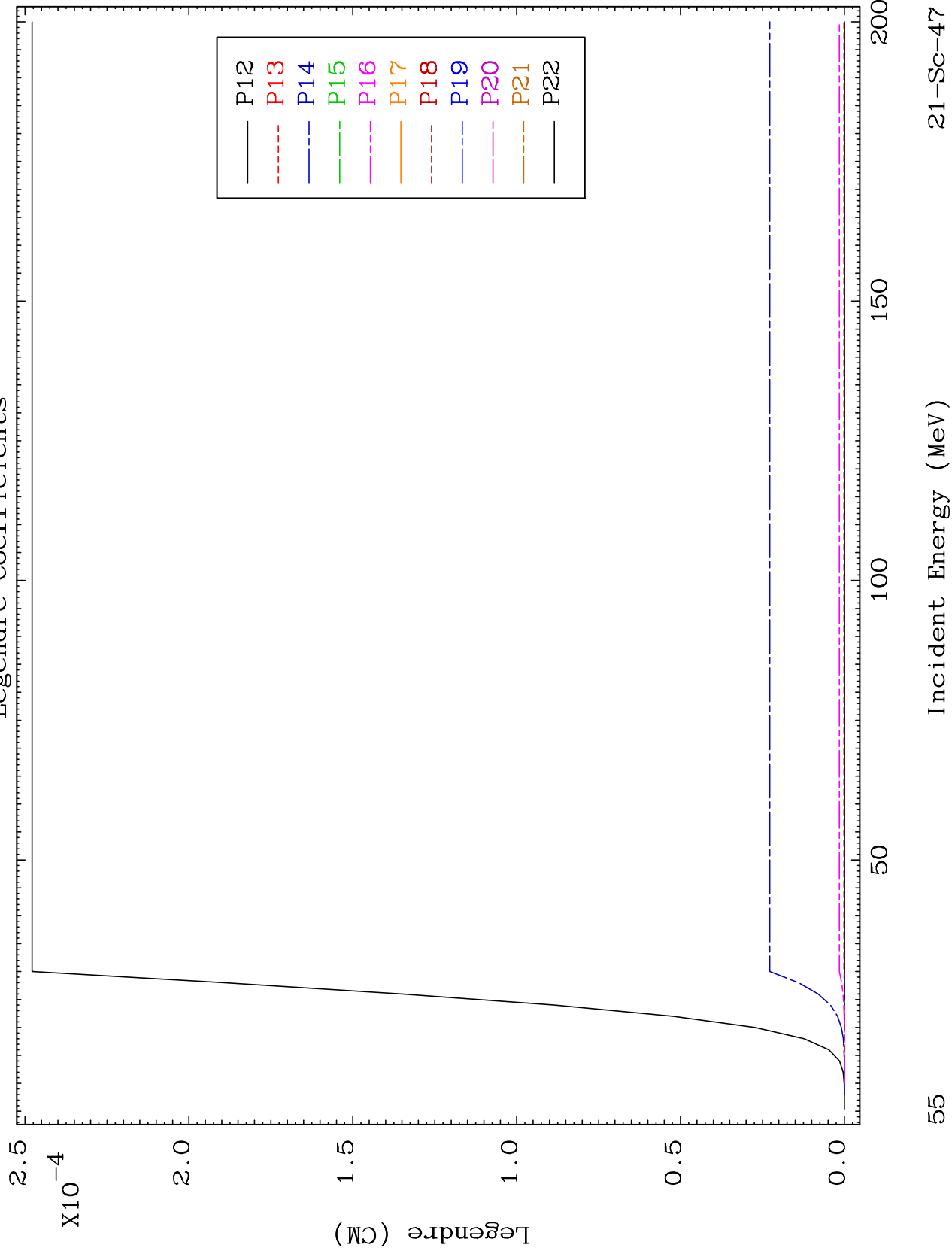




MAT 2131

MT= 64 (n,n') Level
Legendre Coefficients

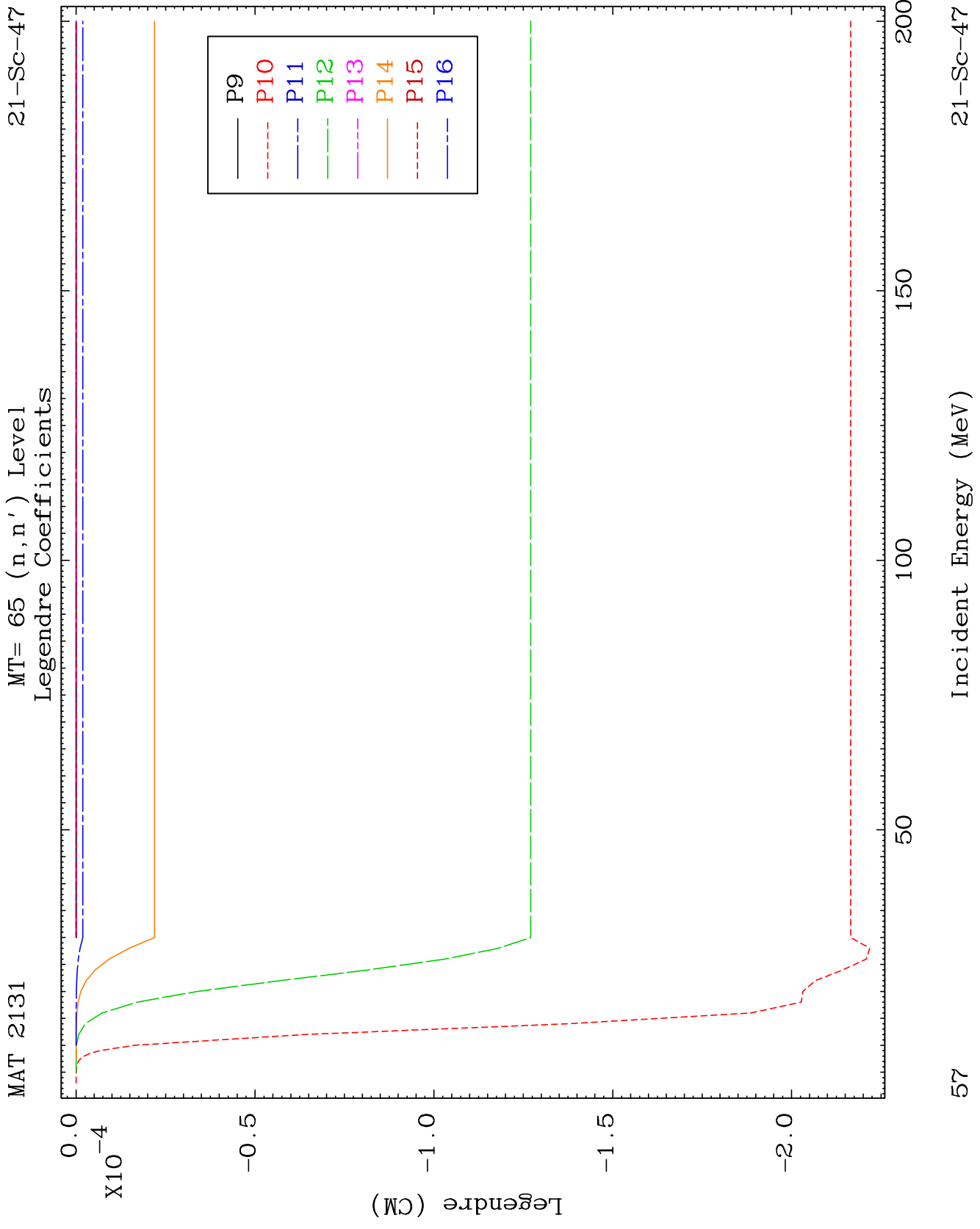
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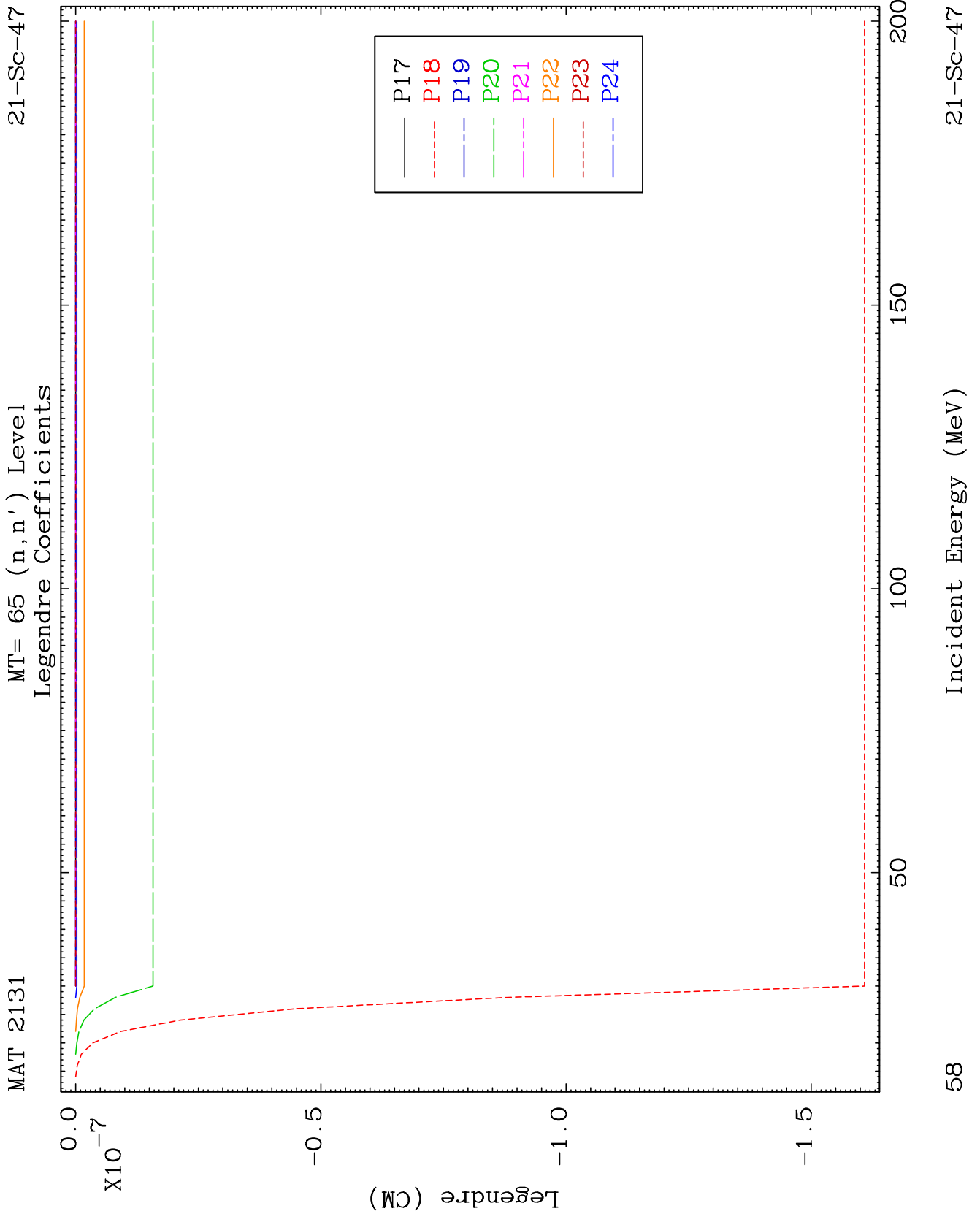


55

Incident Energy (MeV)

21-Sc-47

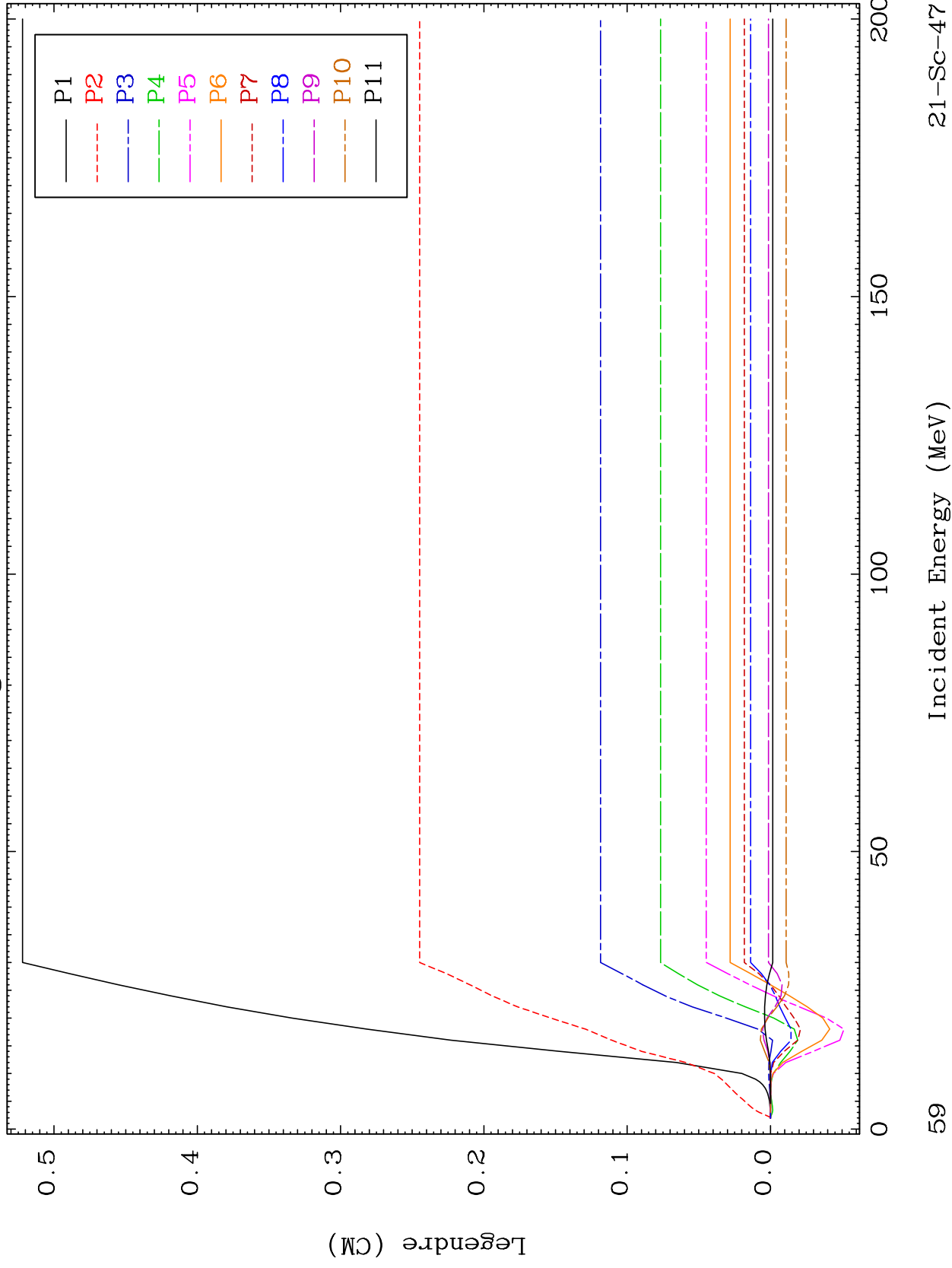




MAT 2131

MT= 66 (n,n') Level
Legendre Coefficients

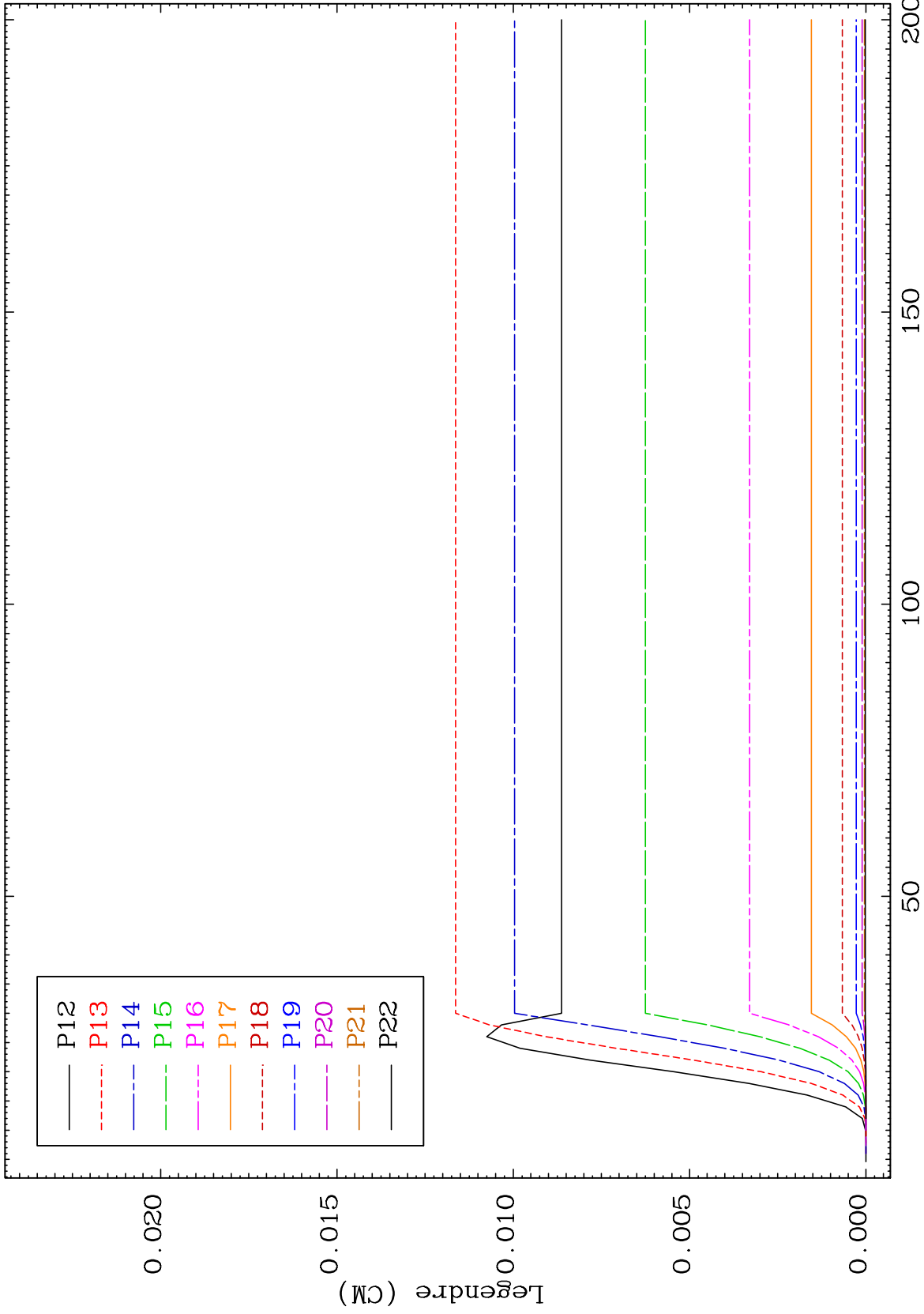
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MAT 2131

MT= 66 (n,n') Level
Legendre Coefficients

21-Sc-47



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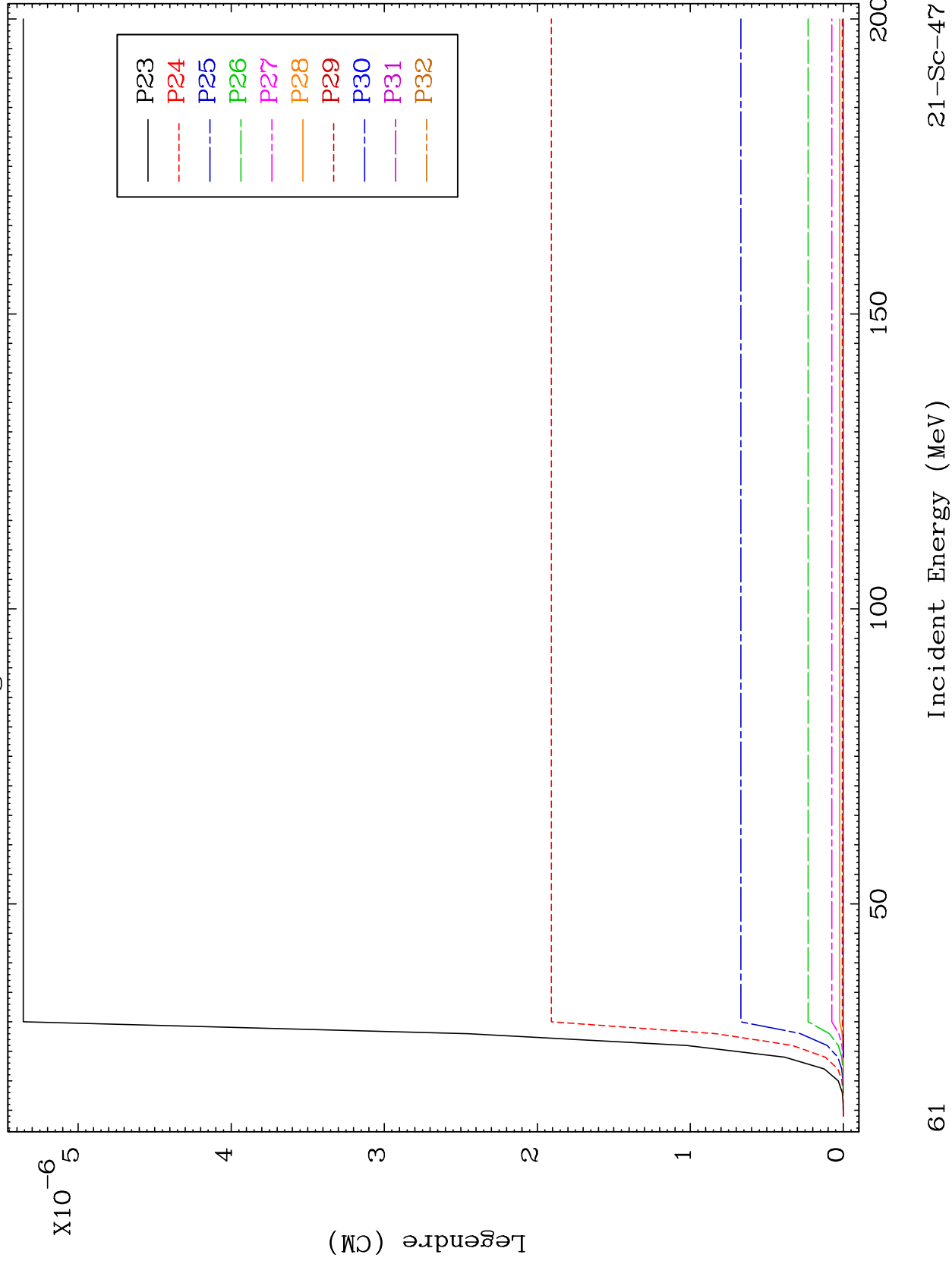
Incident Energy (MeV)

21-Sc-47

MAT 2131

MT= 66 (n,n') Level
Legendre Coefficients

21-Sc-47

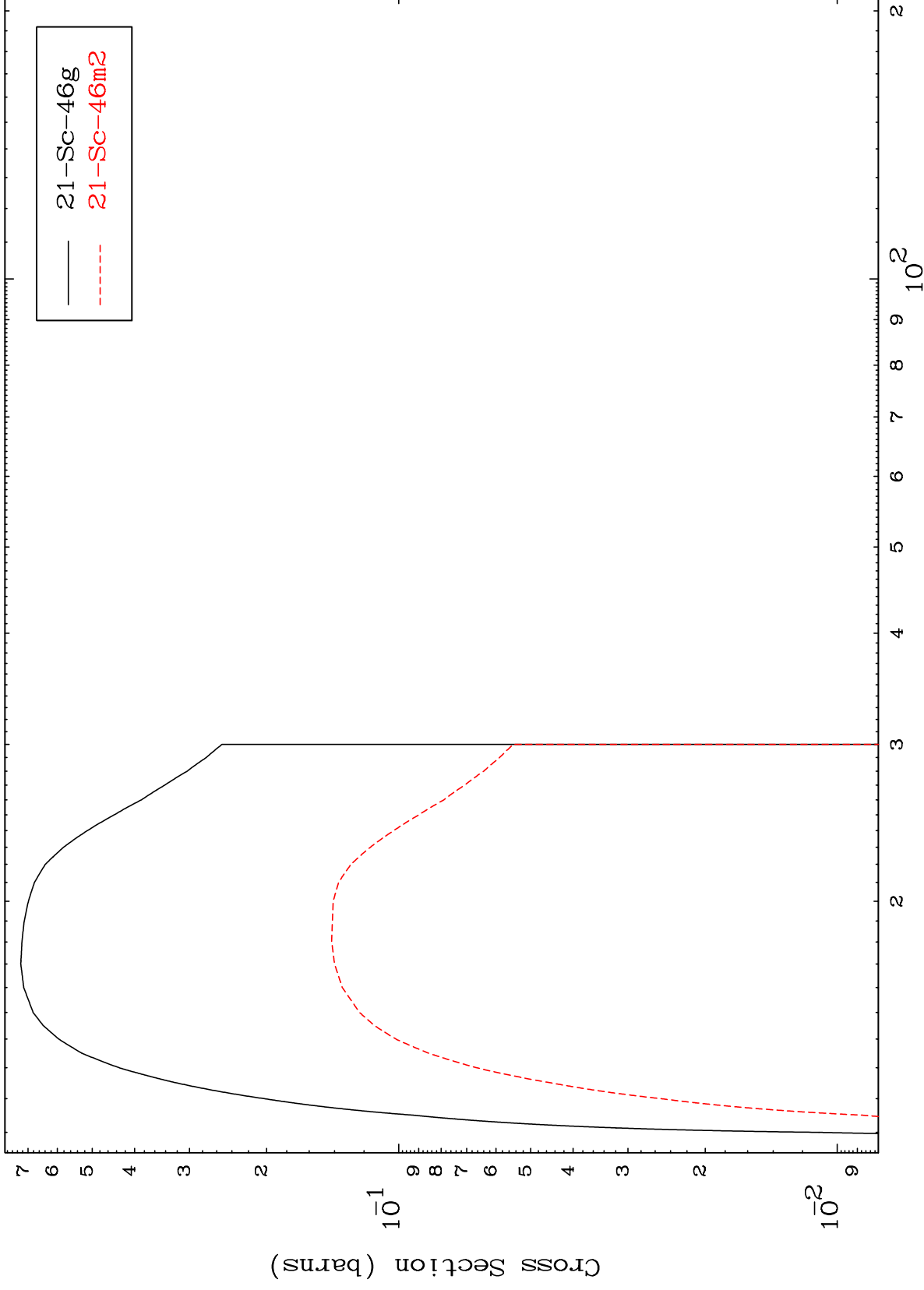


MAT 2131

(n,2n)

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Radionuclide Production Cross Section



62

Incident Energy (MeV)

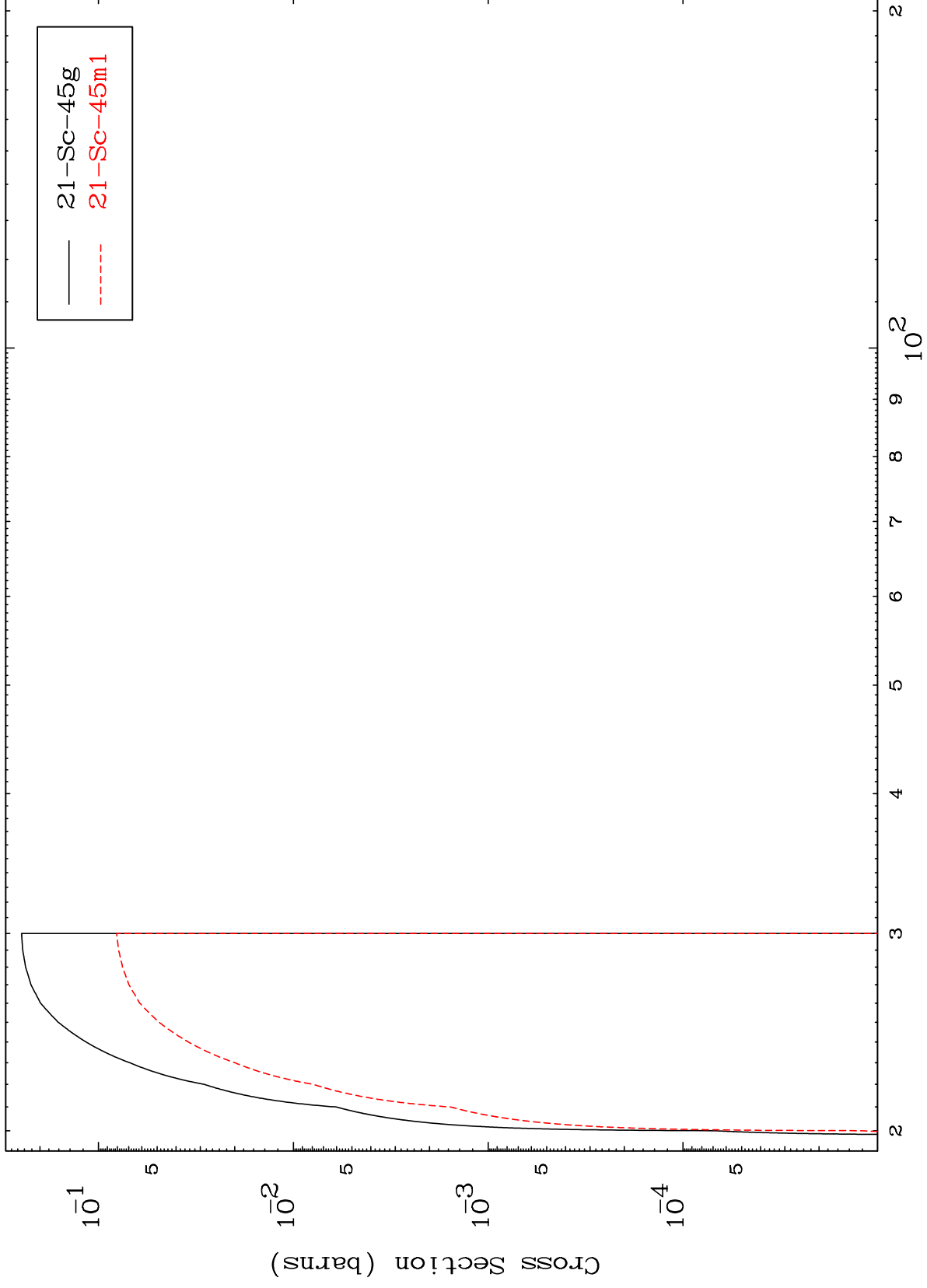
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MAT 2131

(n,3n)

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Radionuclide Production Cross Section



63

Incident Energy (MeV)

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