

Program EVALPLOT
(Version 2021-1)

by

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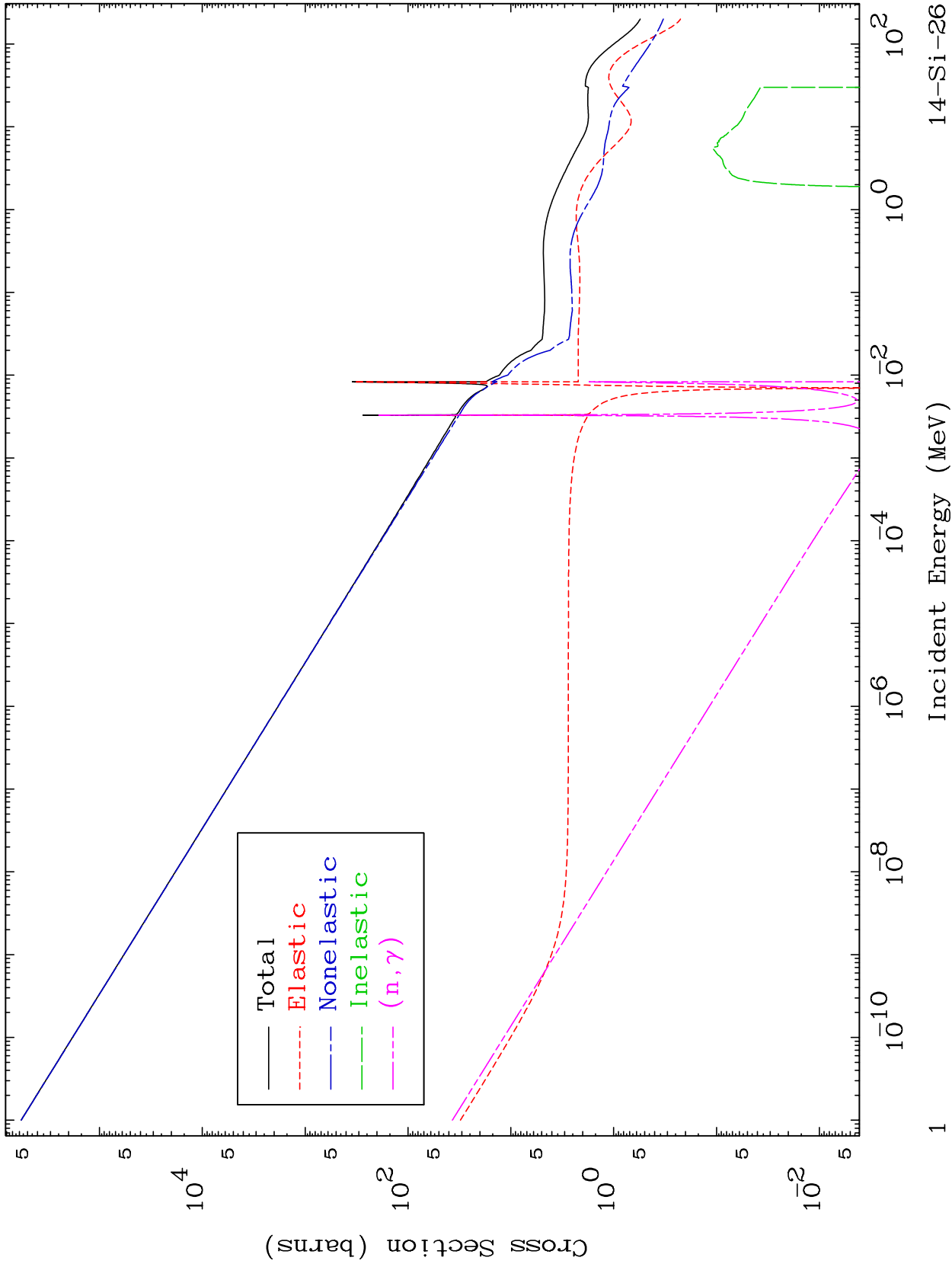
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 1419

Neutron Major
293 Kelvin Cross Sections

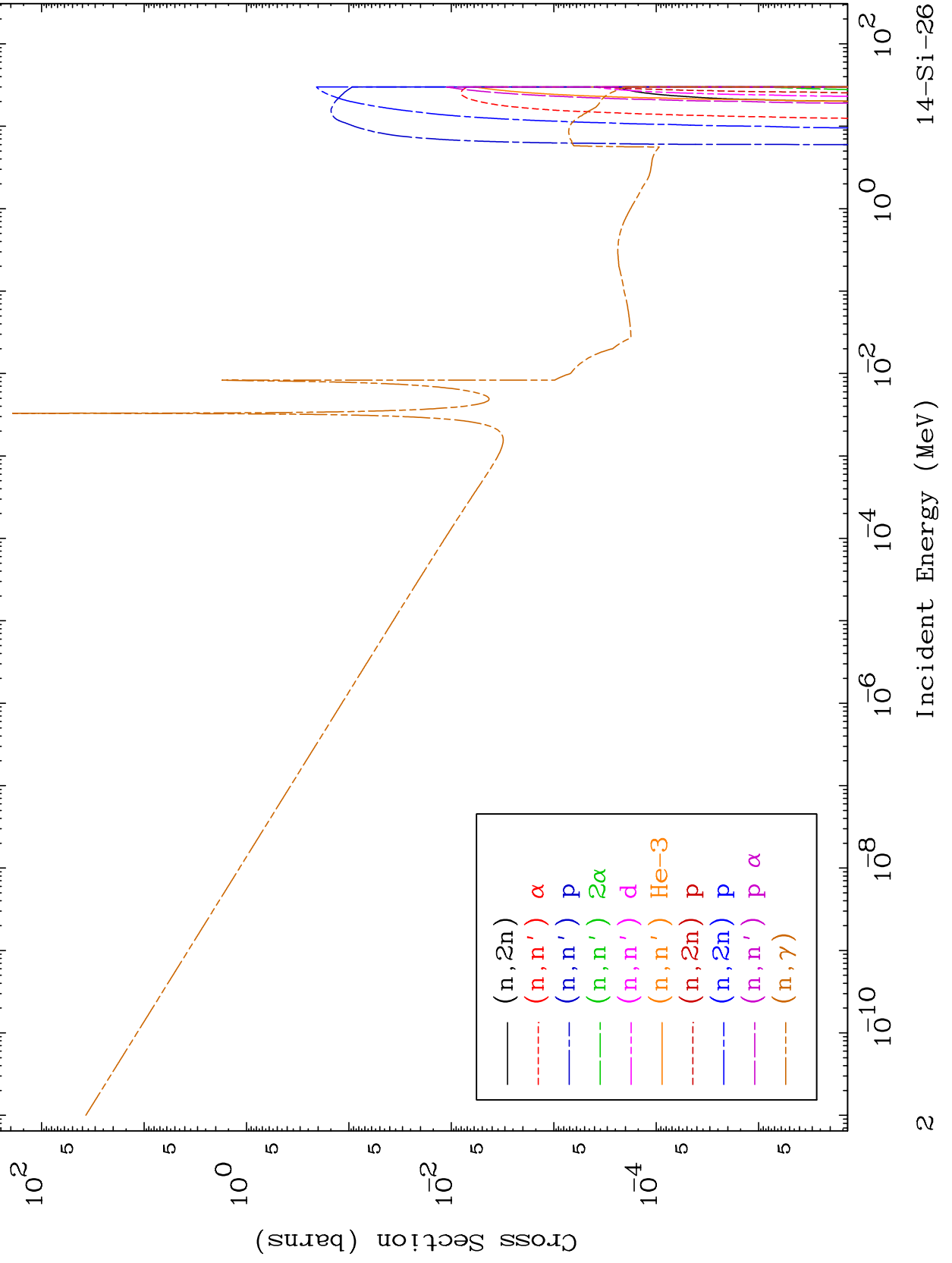
14-Si-26



MAT 1419

Neutron Absorption
293 Kelvin Cross Sections

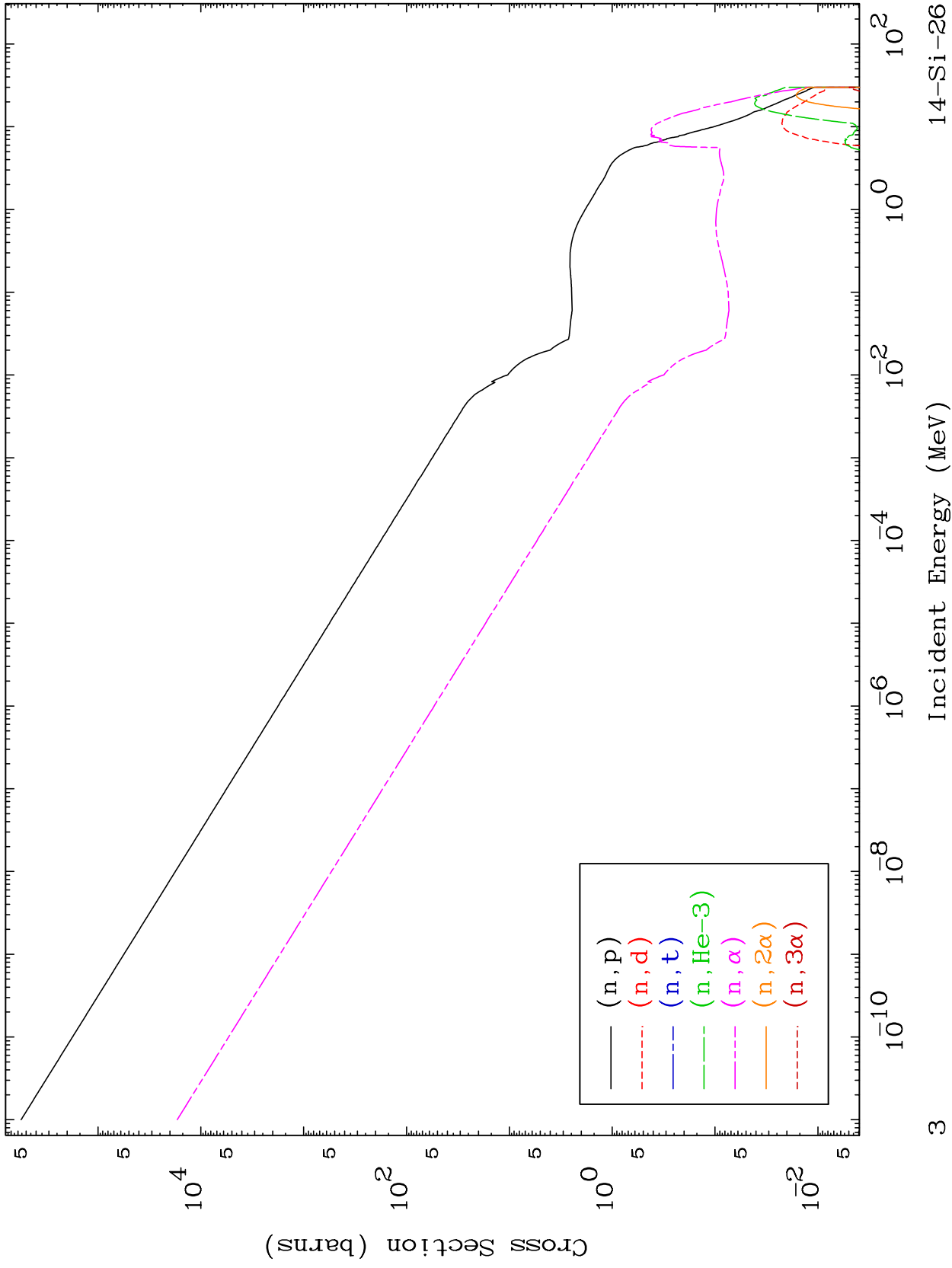
14-Si-26



MAT 1419

Neutron Absorption
293 Kelvin Cross Sections

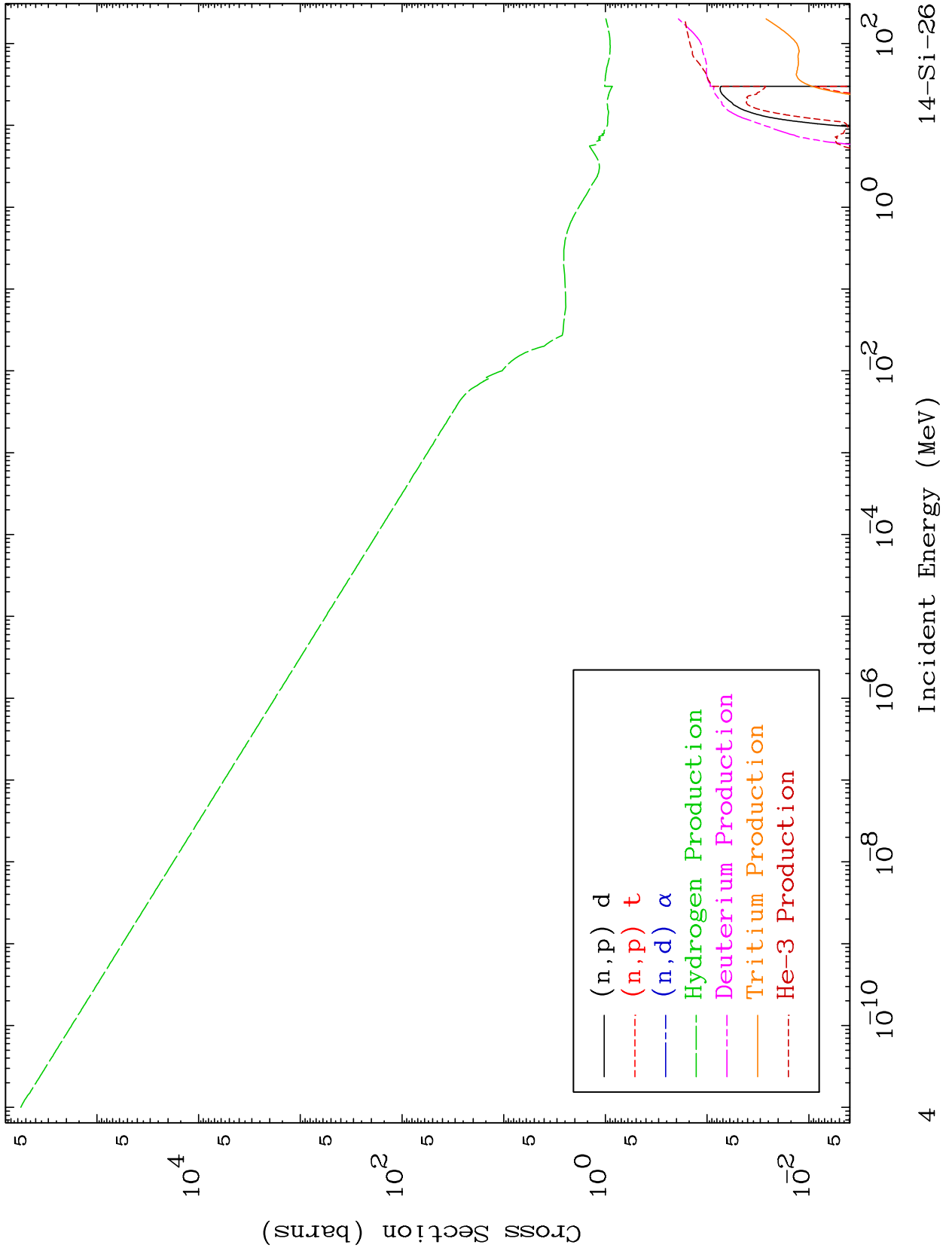
14-Si-26



MAT 1419

Neutron Absorption
293 Kelvin Cross Sections

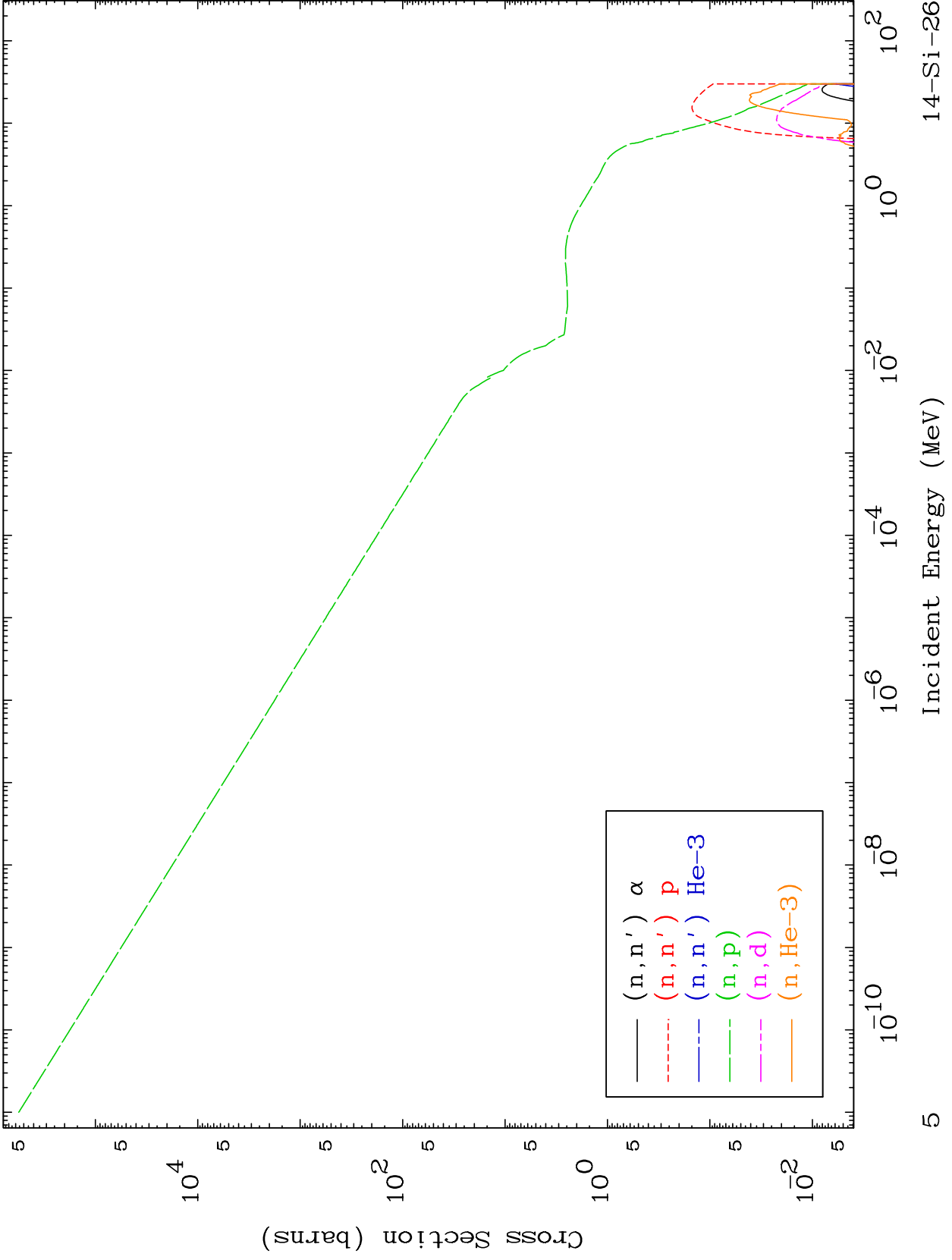
14-Si-26



MAT 1419

Charged Particle
293 Kelvin Cross Sections

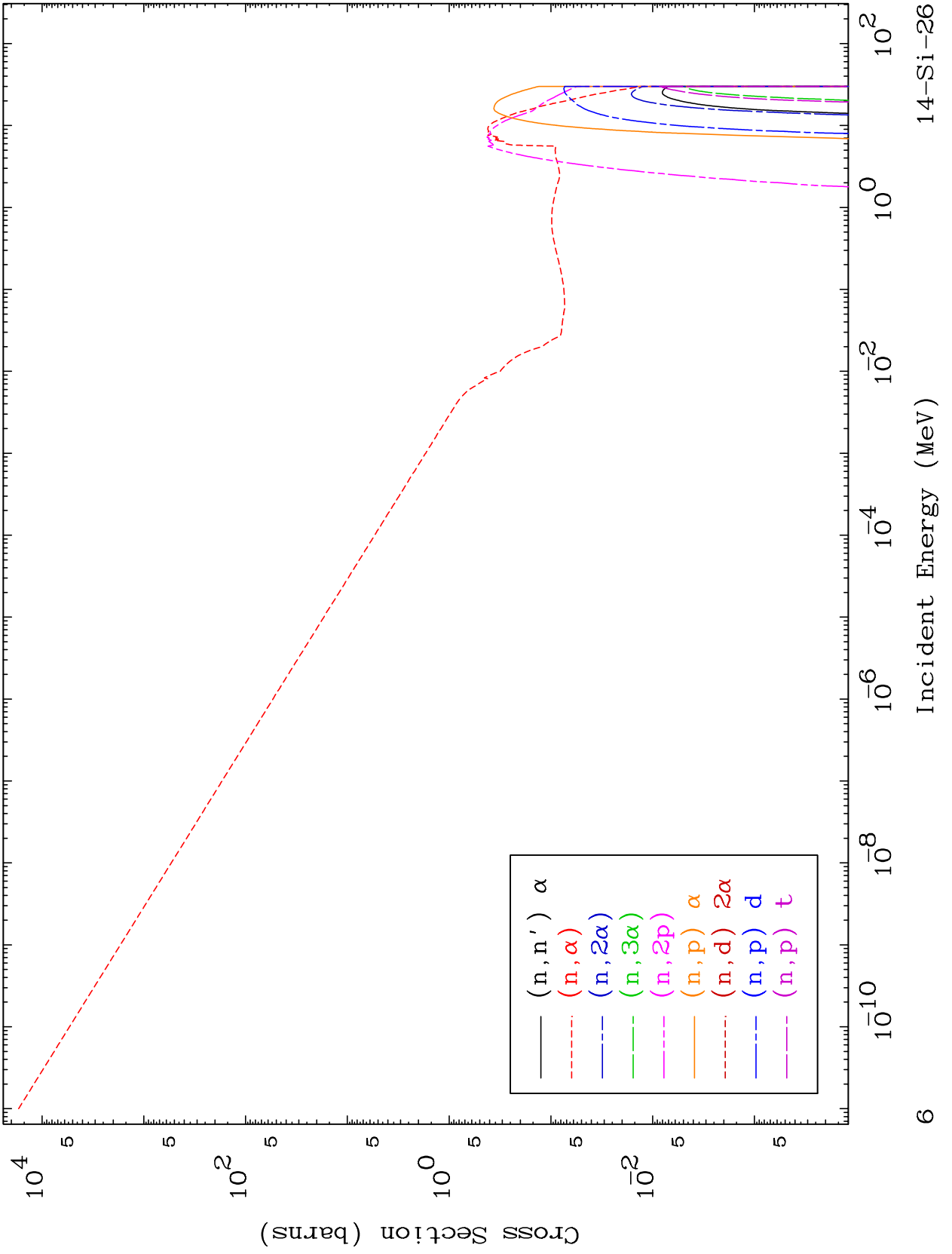
14-Si-26



MAT 1419

Charged Particle
293 Kelvin Cross Sections

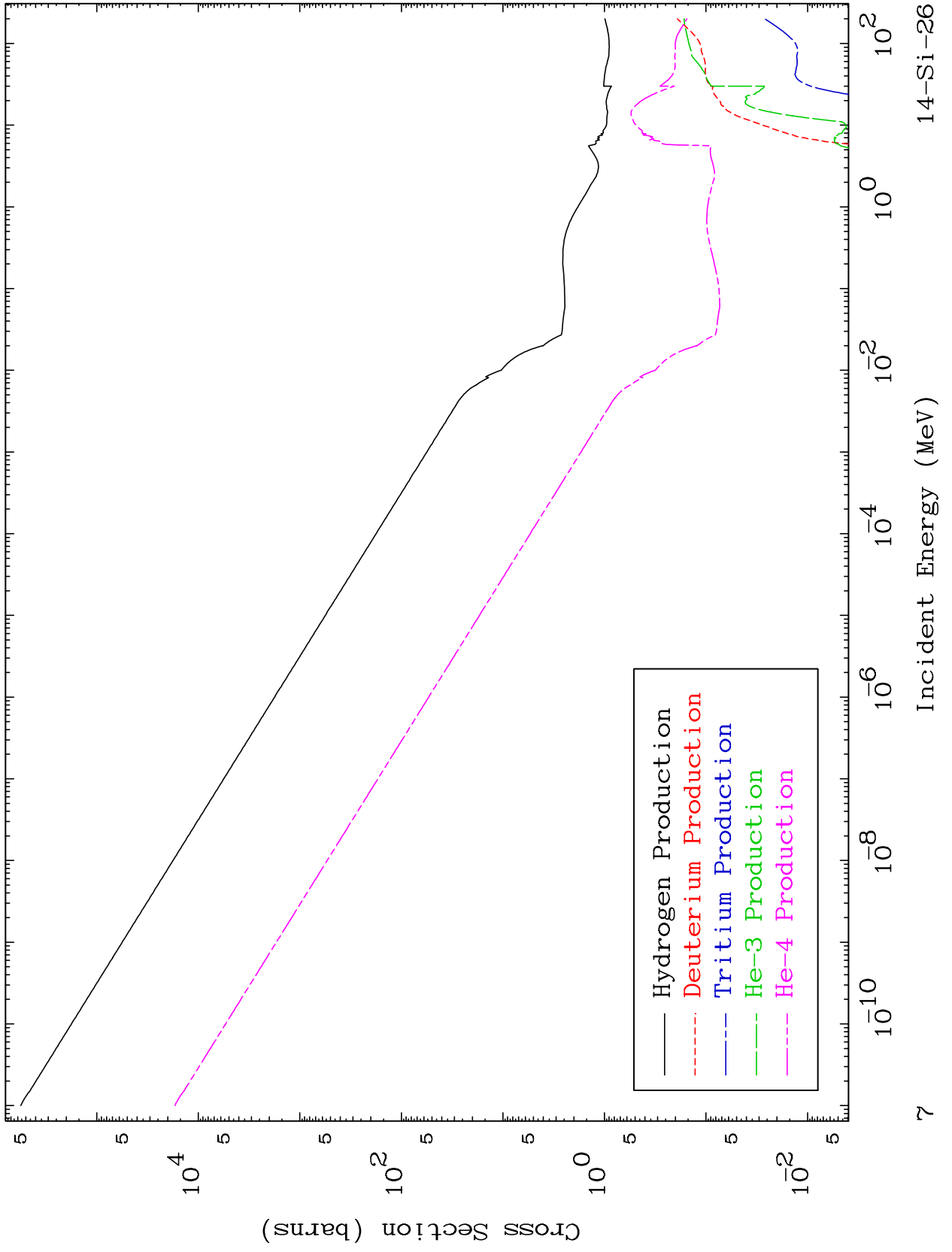
14-Si-26



MAT 1419

Particle Production
293 Kelvin Cross Sections

14-Si-26

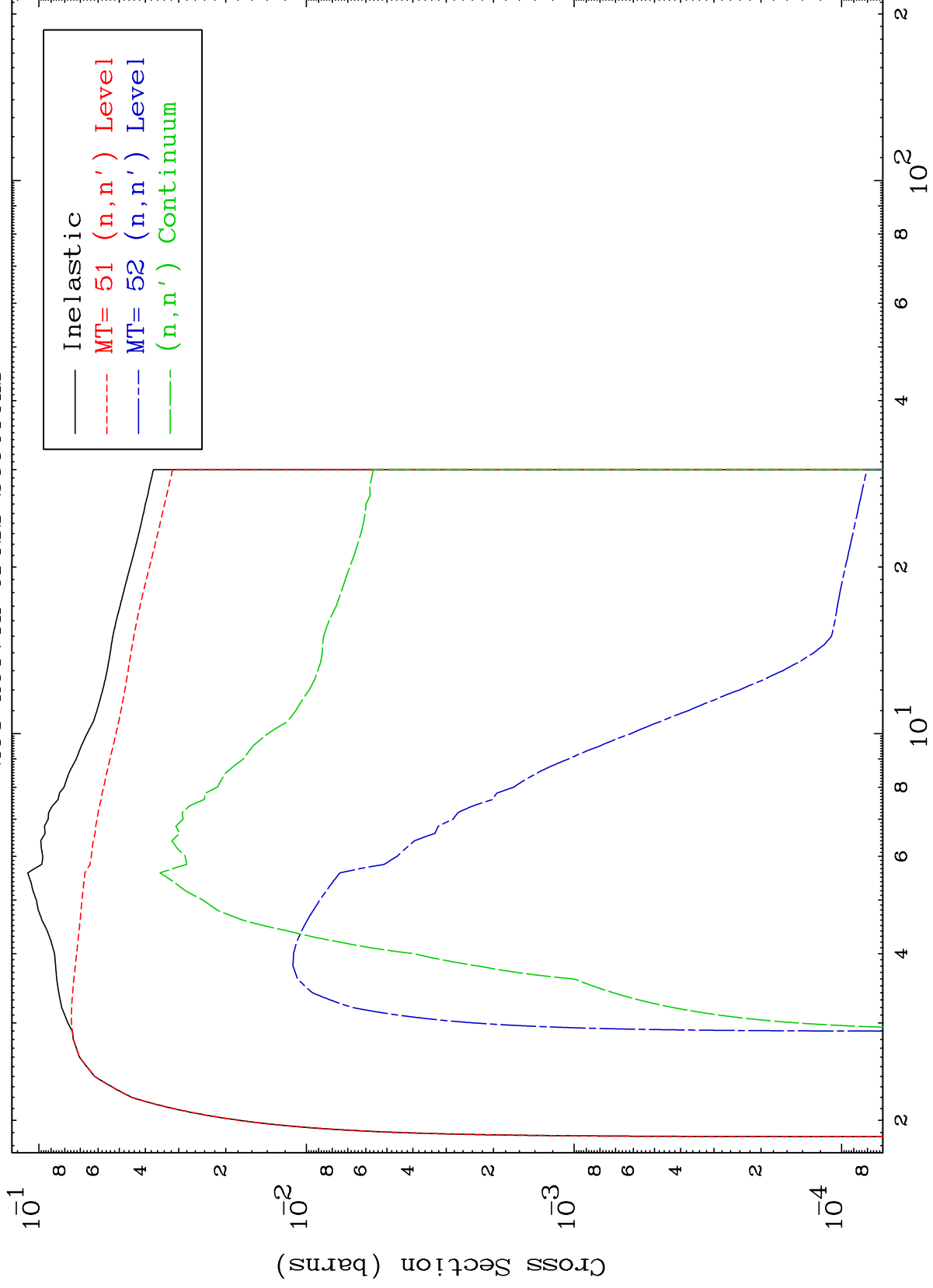


MAT 1419

(n,n') Levels

293 Kelvin Cross Sections

14-Si-26



— Inelastic
- - - $MT=51$ (n,n') Level
- - - $MT=52$ (n,n') Level
- - - (n,n') Continuum

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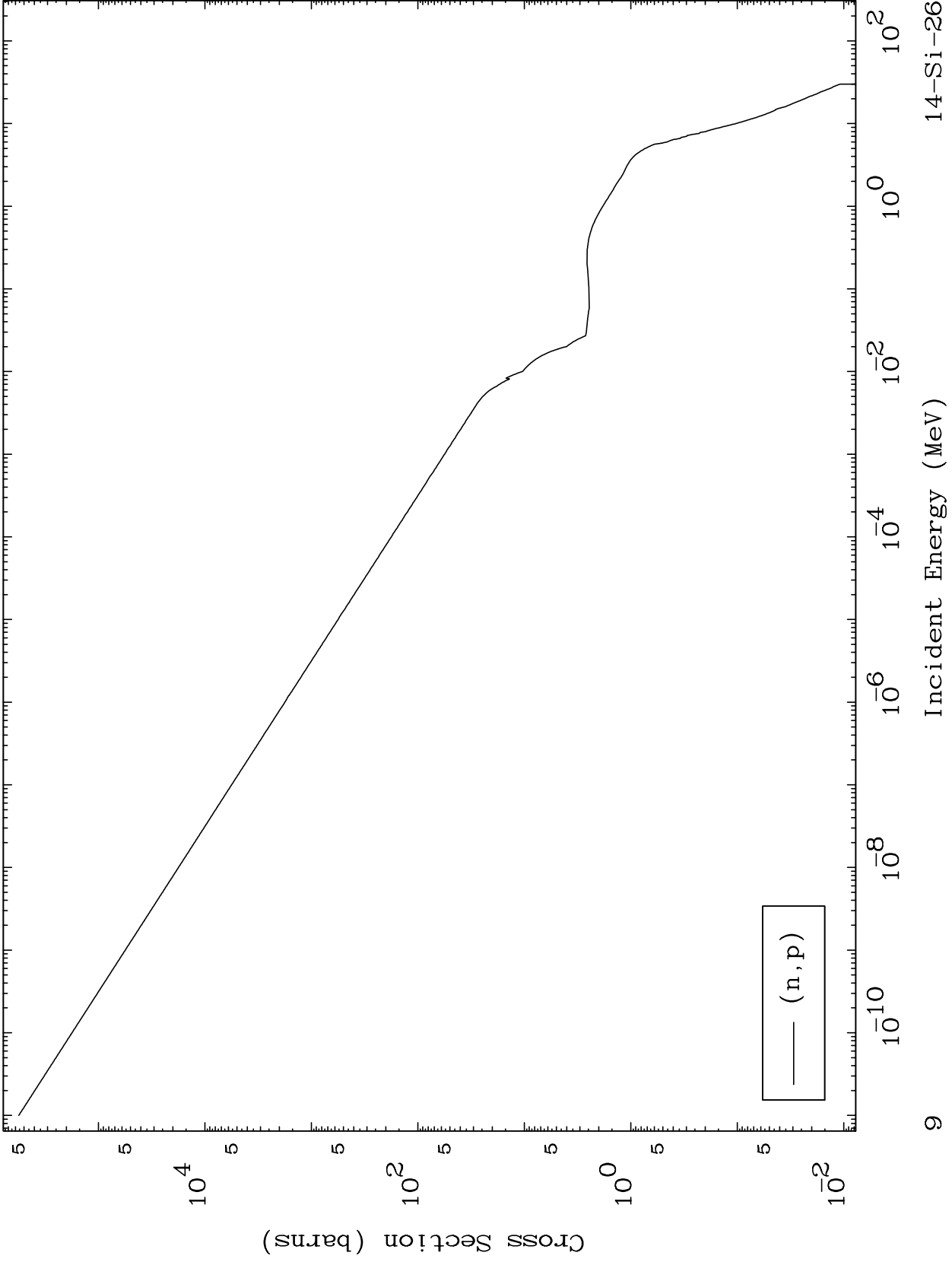
Incident Energy (MeV)

14-Si-26

MAT 1419

(n,p) Levels
293 Kelvin Cross Sections

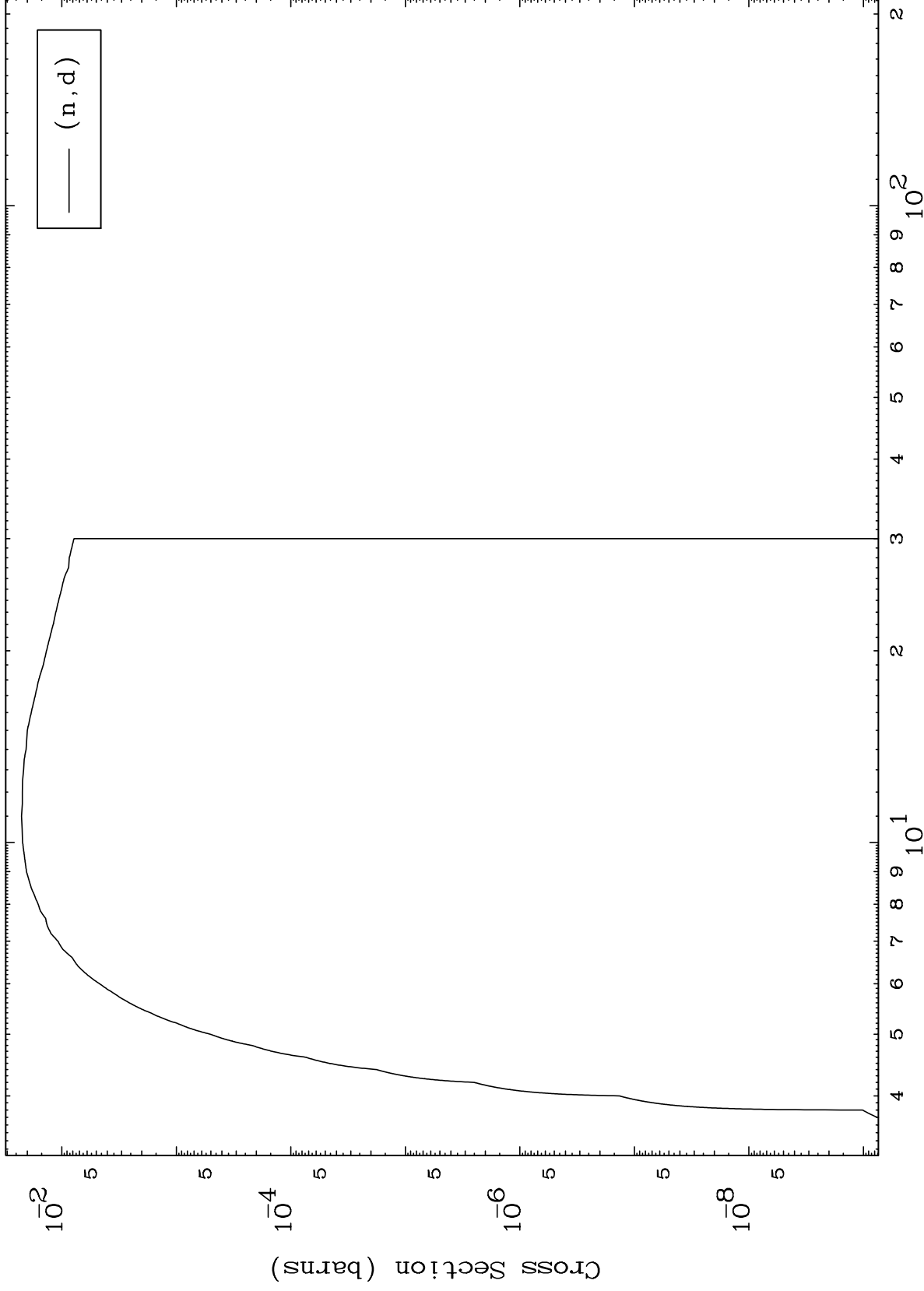
14-Si-26



MAT 1419

(n,d) Levels
293 Kelvin Cross Sections

14-Si-26



10

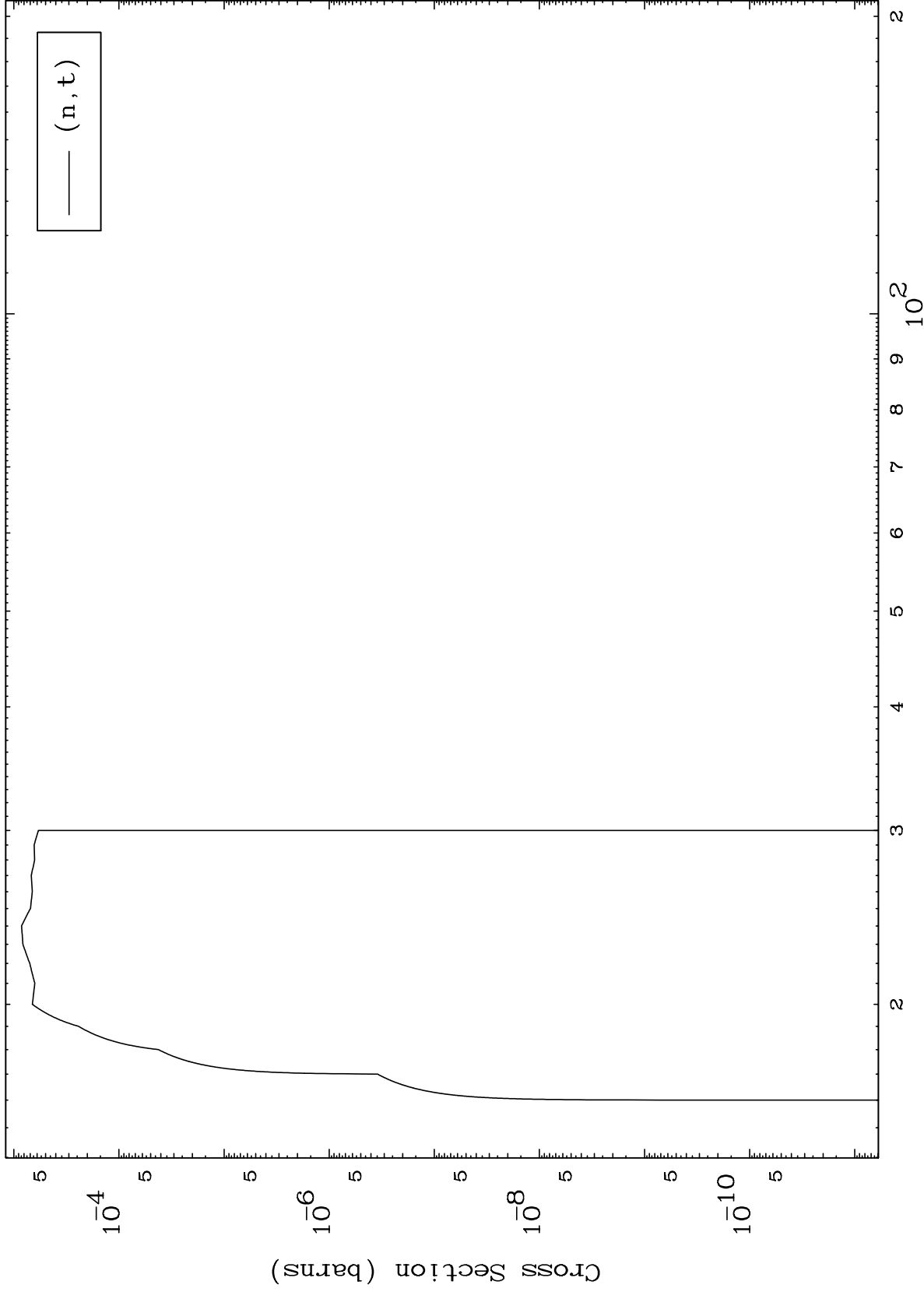
Incident Energy (MeV)

14-Si-26

MAT 1419

(n,t) Levels
293 Kelvin Cross Sections

14-Si-26



11

Incident Energy (MeV)

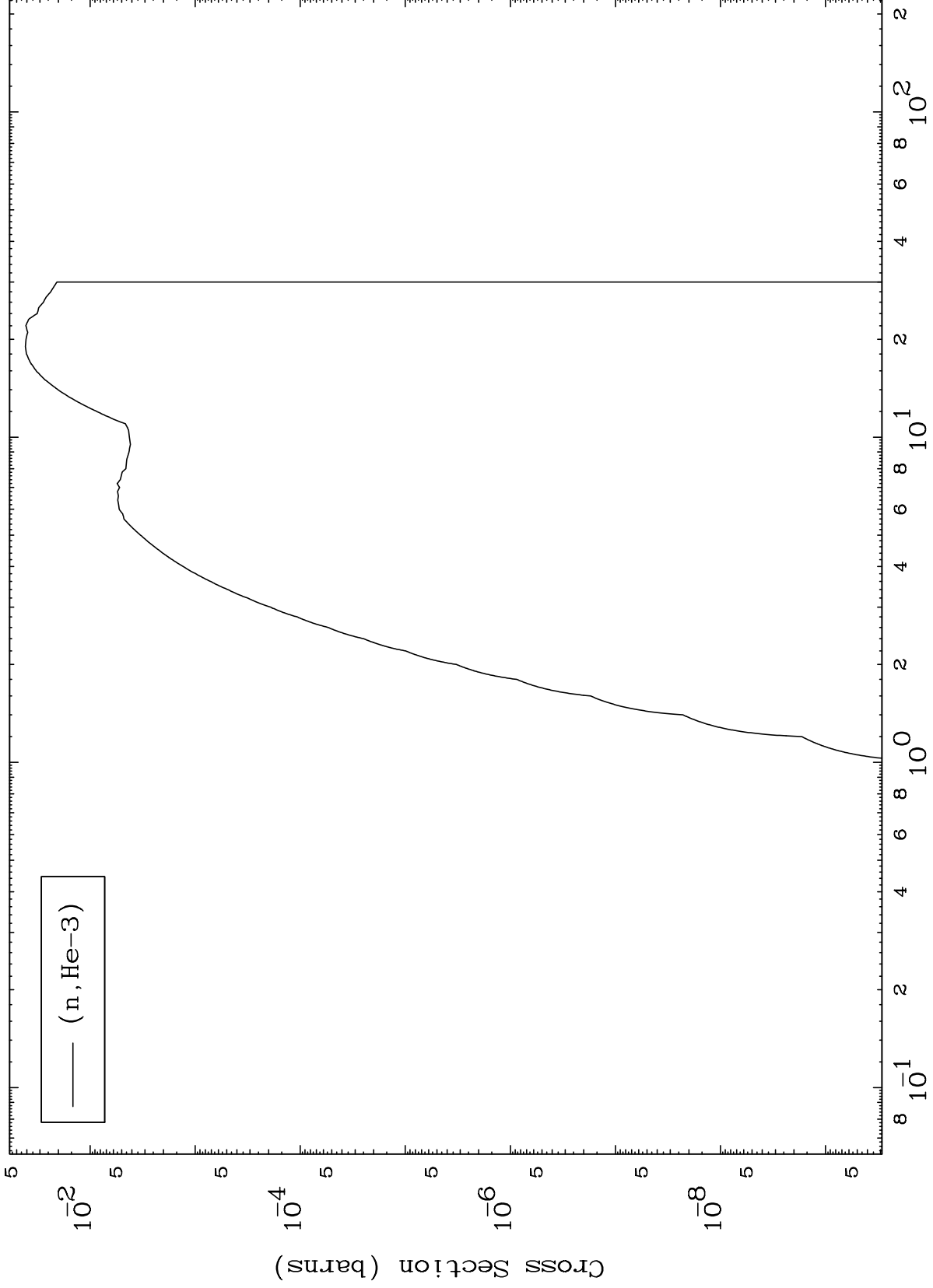
14-Si-26

MAT 1419

(n,He3) Levels

14-Si-26

293 Kelvin Cross Sections



12

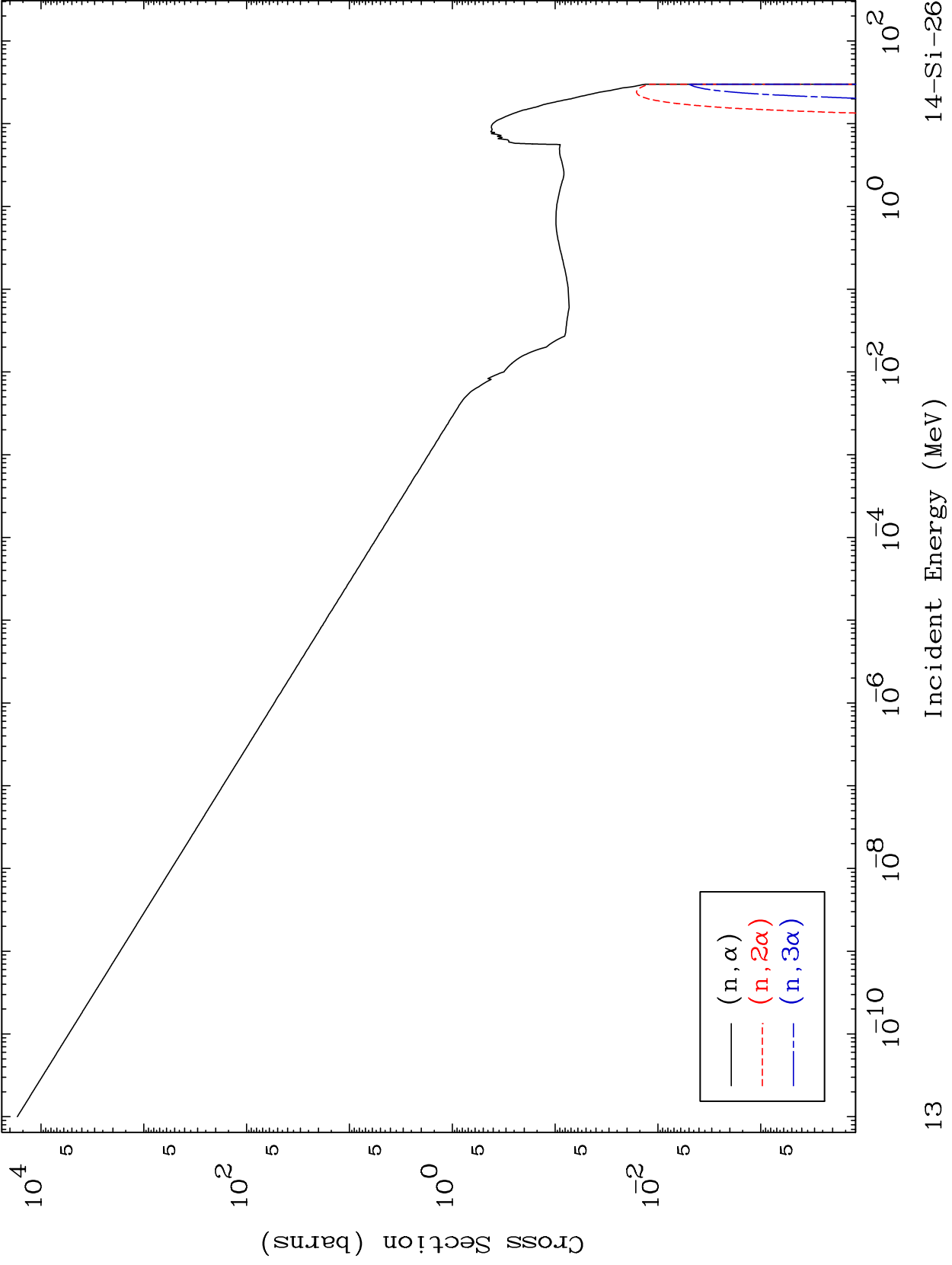
Incident Energy (MeV)

14-Si-26

MAT 1419

(n,α) Levels
293 Kelvin Cross Sections

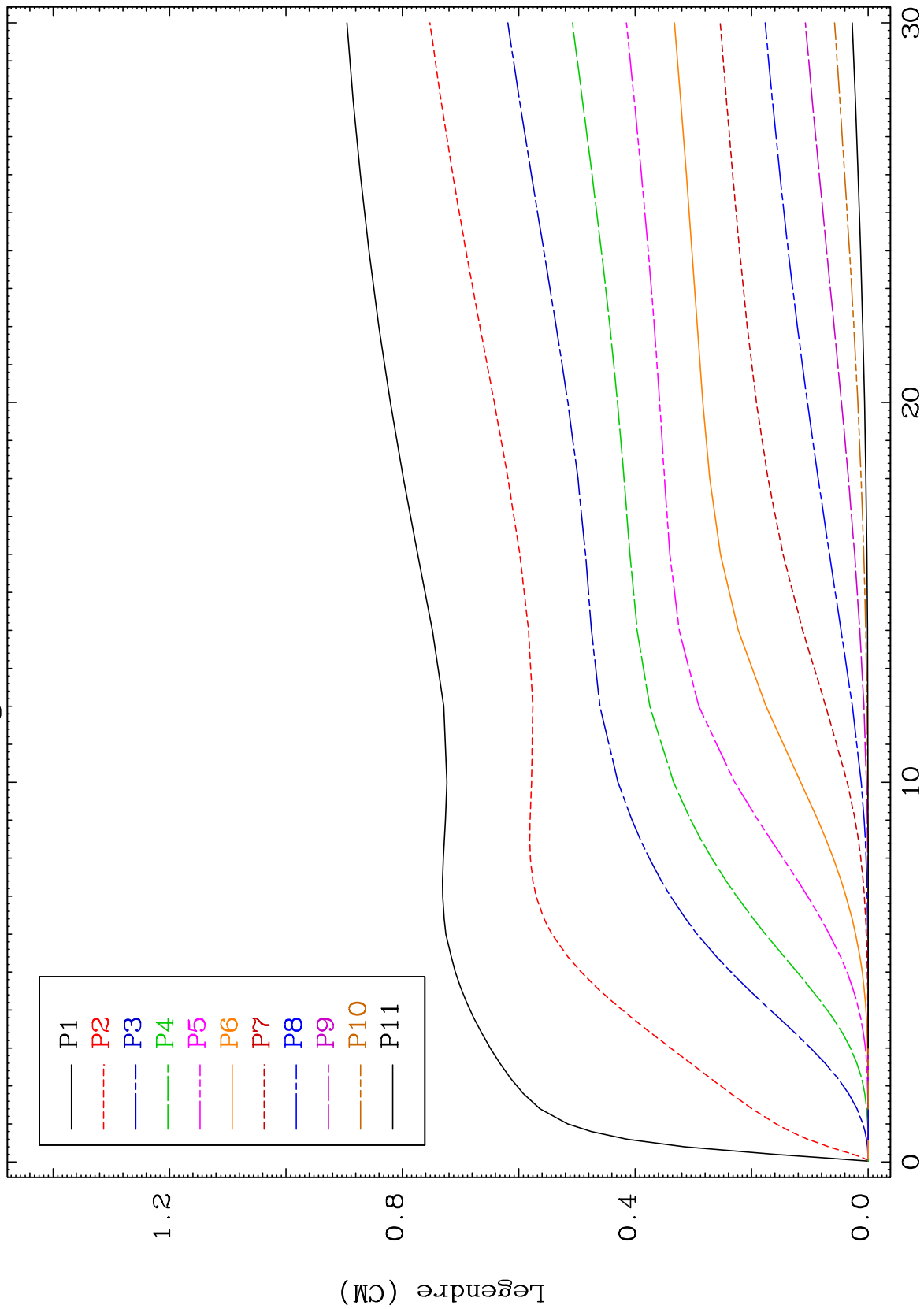
14-Si-26



MAT 1419

Elastic Legendre Coefficients

¹⁴Si-26



14

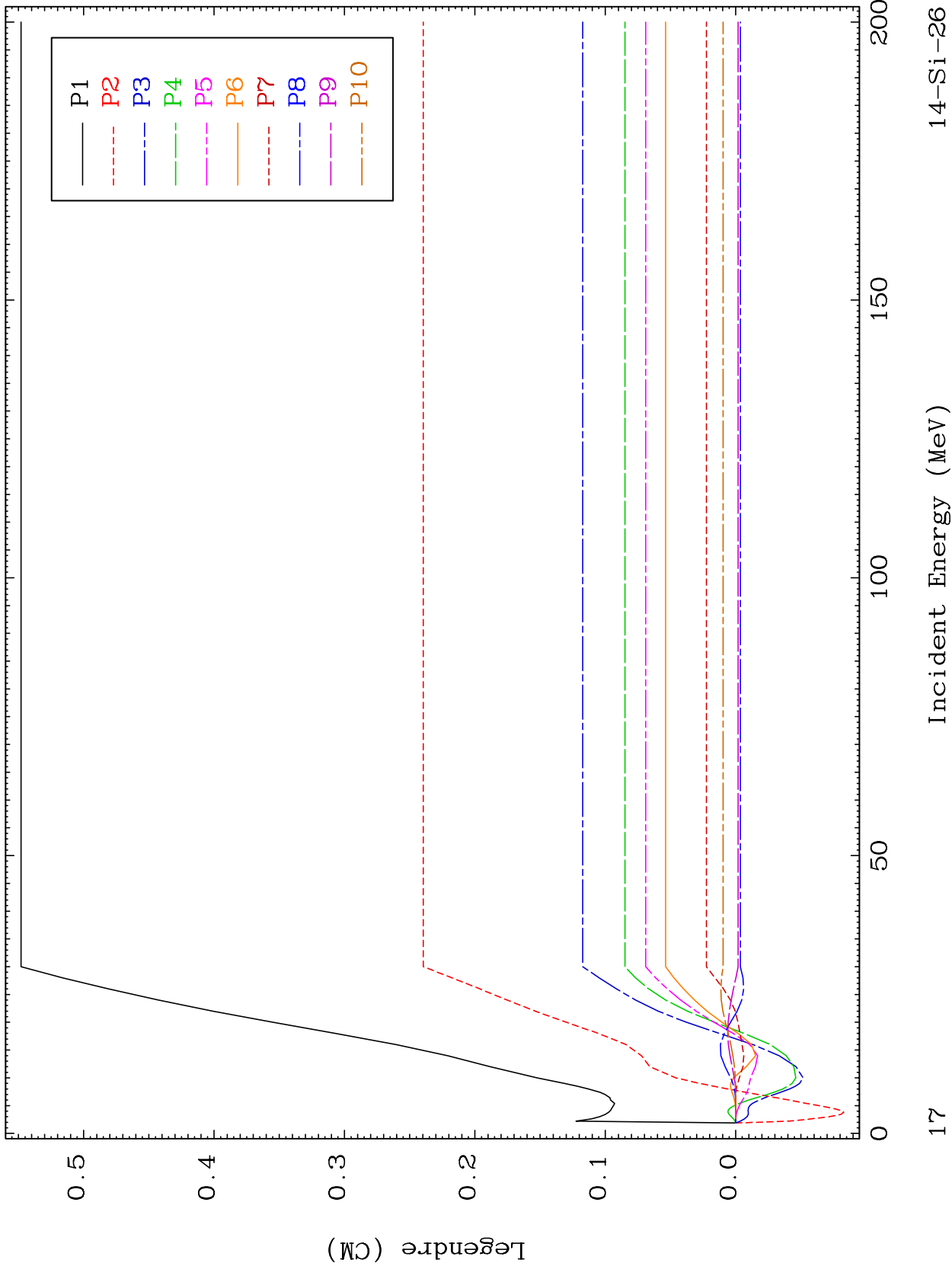
Incident Energy (MeV)

¹⁴Si-26

MAT 1419

MT= 51 (n,n') Level
Legendre Coefficients

14-Si-26



14-Si-26

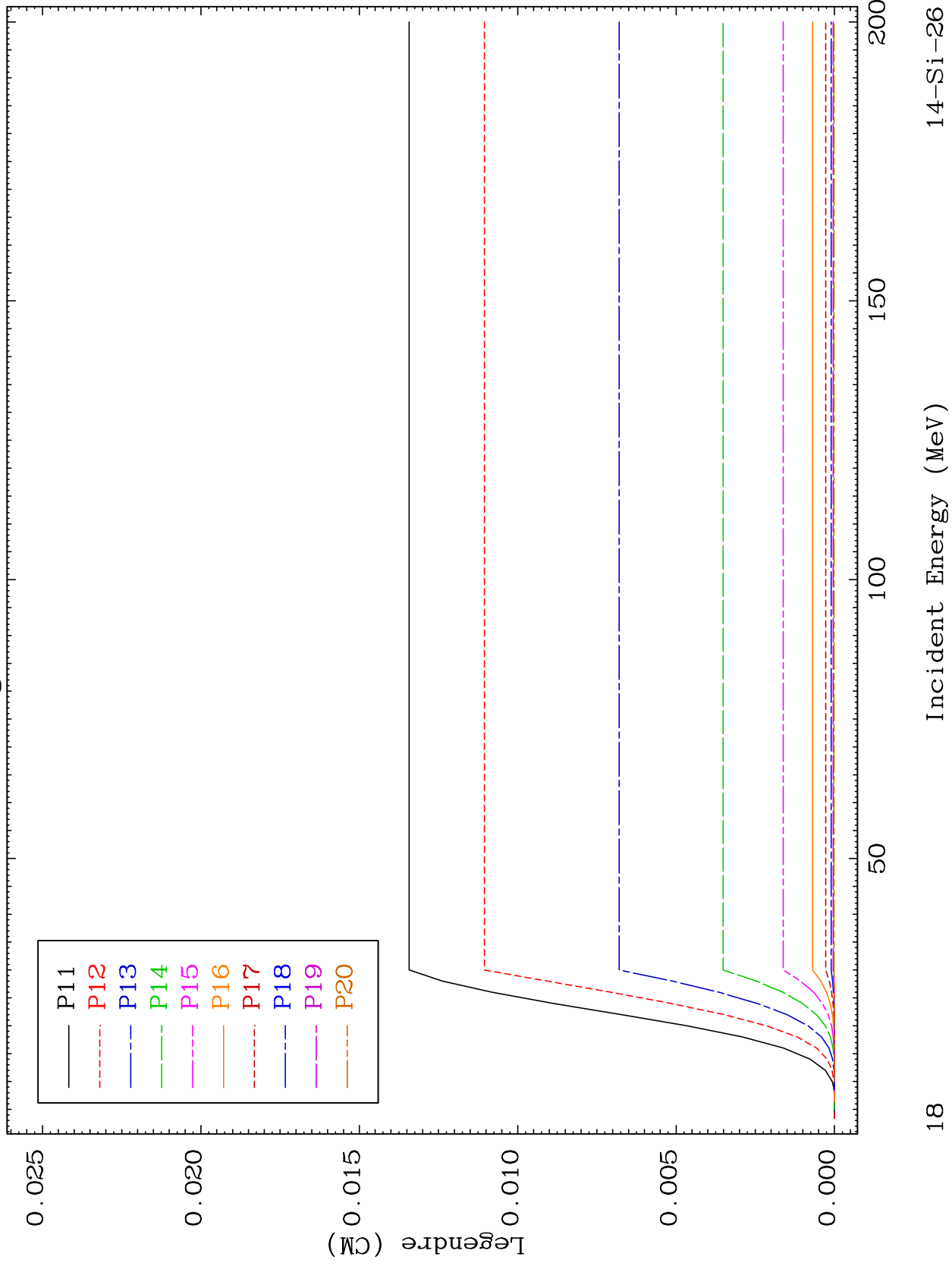
Incident Energy (MeV)

17

MAT 1419

MT= 51 (n,n') Level
Legendre Coefficients

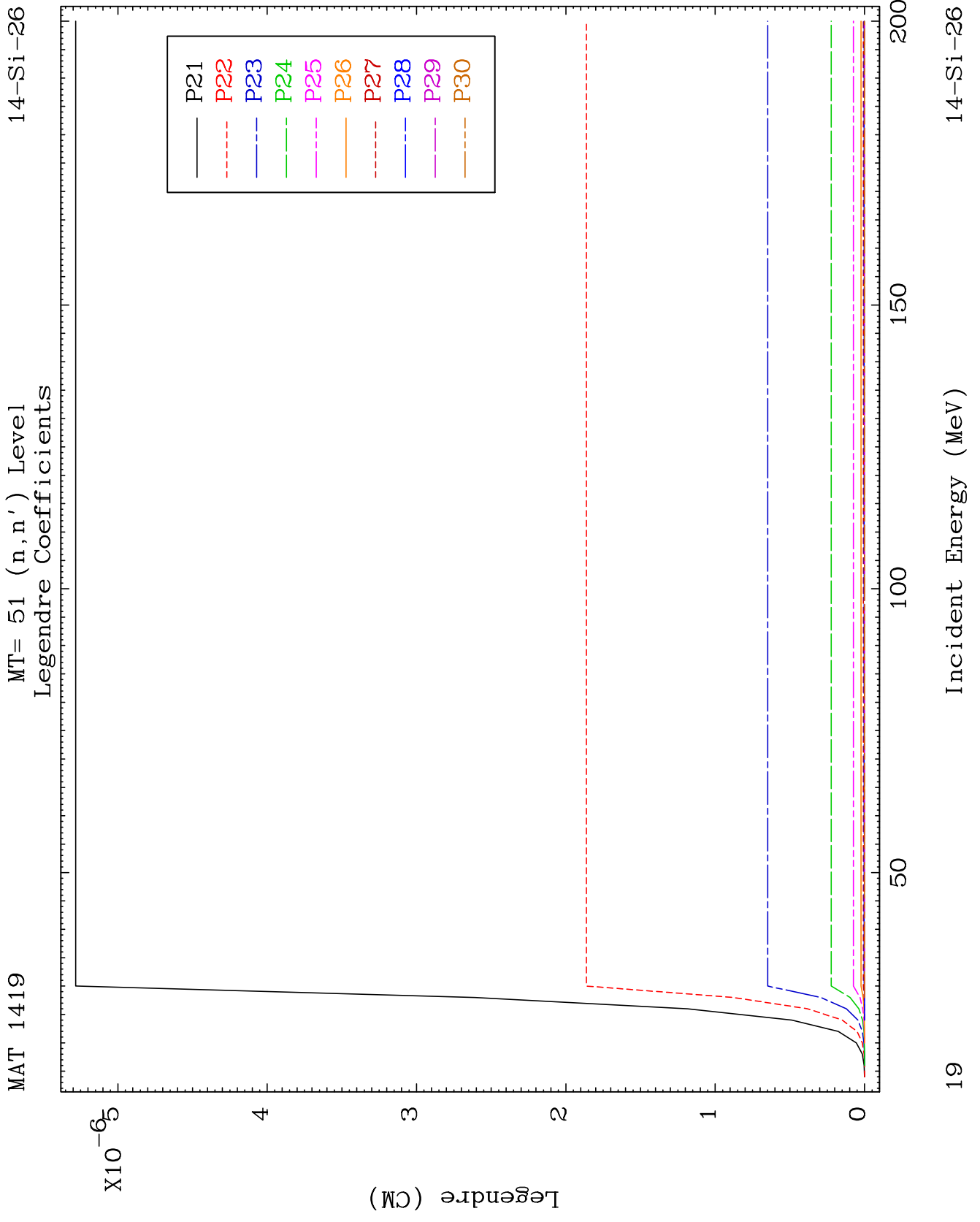
14-Si-26



18

Incident Energy (MeV)

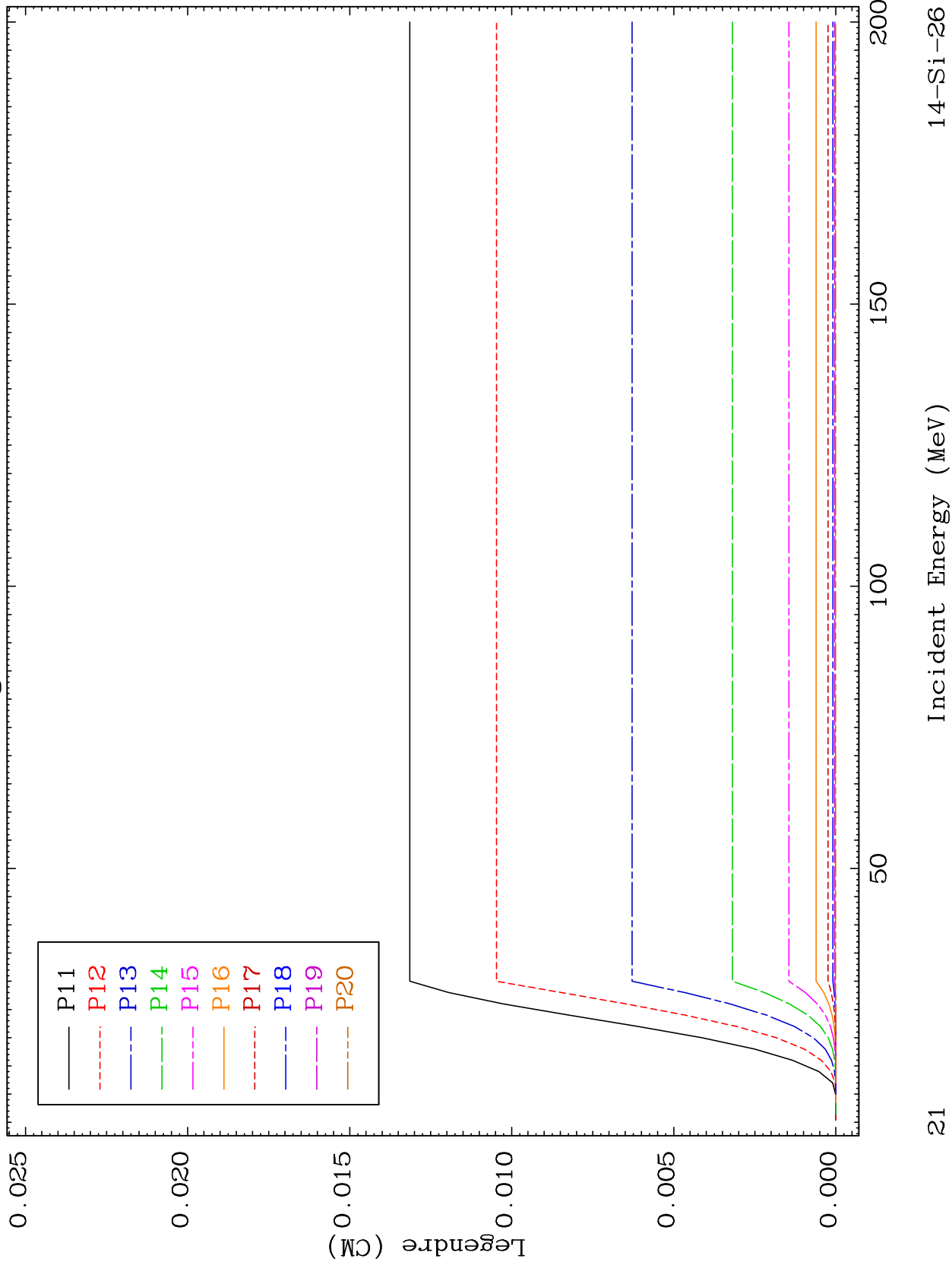
14-Si-26



MAT 1419

MT= 52 (n,n') Level
Legendre Coefficients

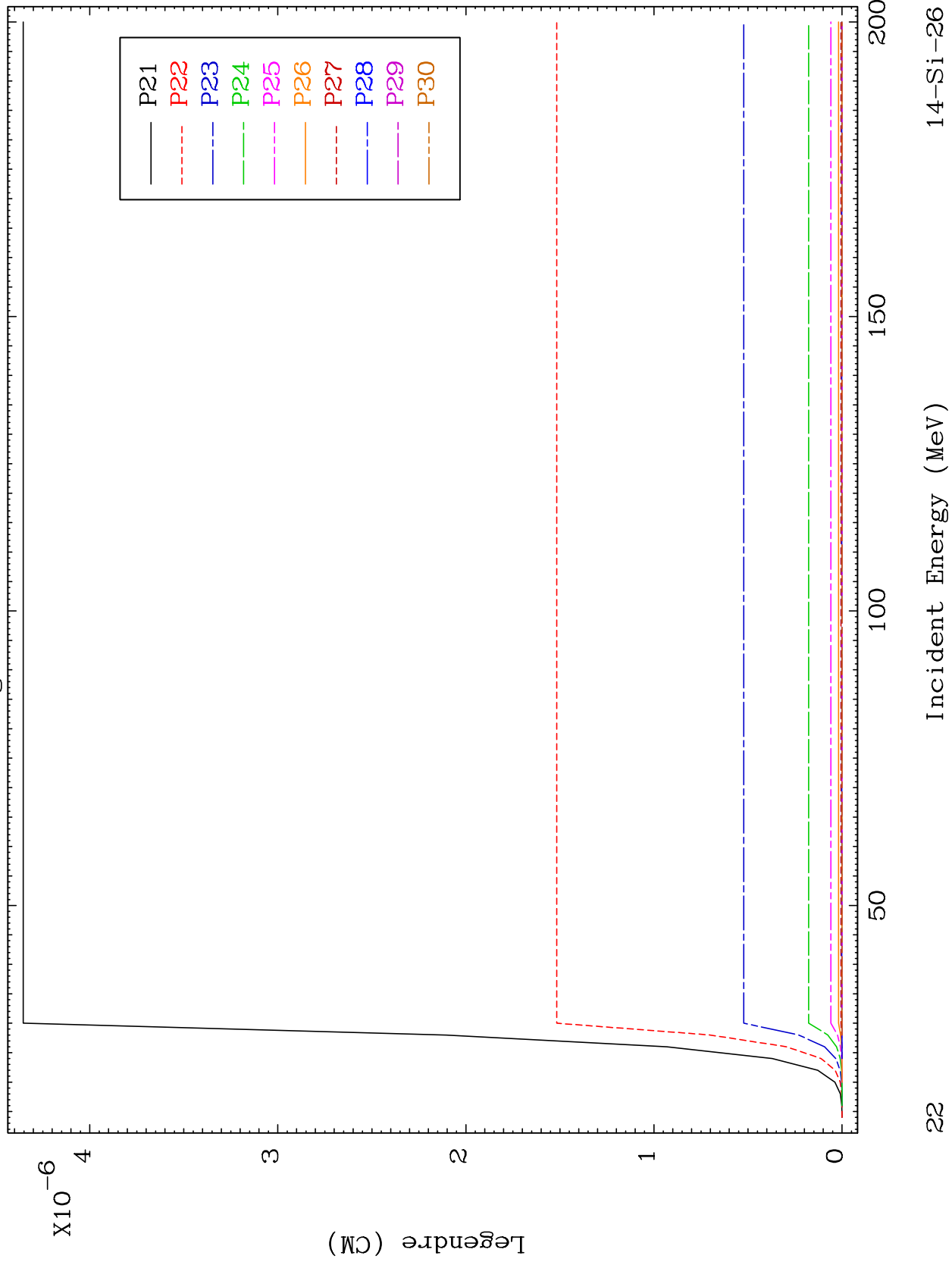
14-Si-26



MAT 1419

MT= 52 (n,n') Level
Legendre Coefficients

14-Si-26

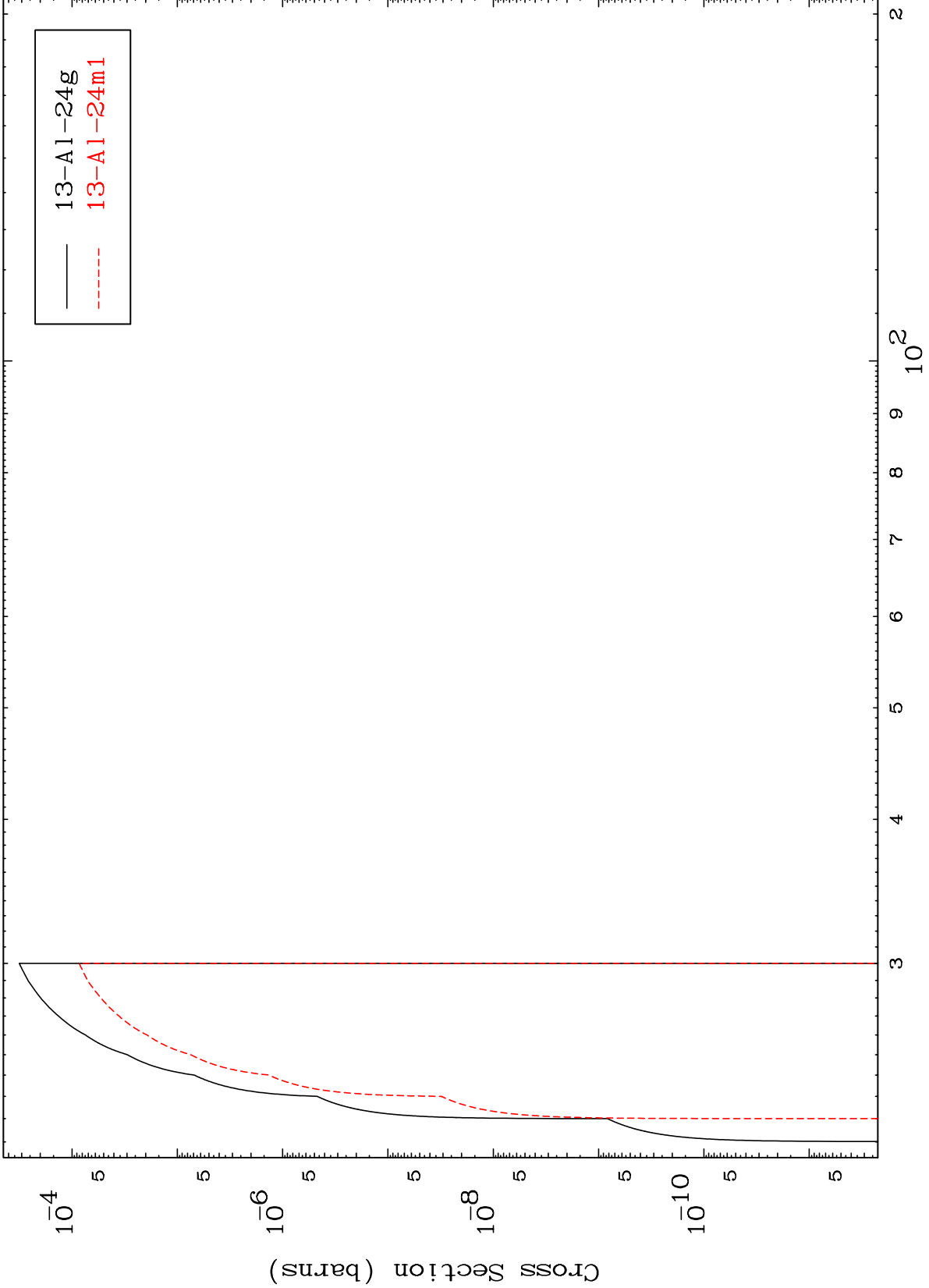


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Incident Energy (MeV)

14-Si-26

Radionuclide Production Cross Section



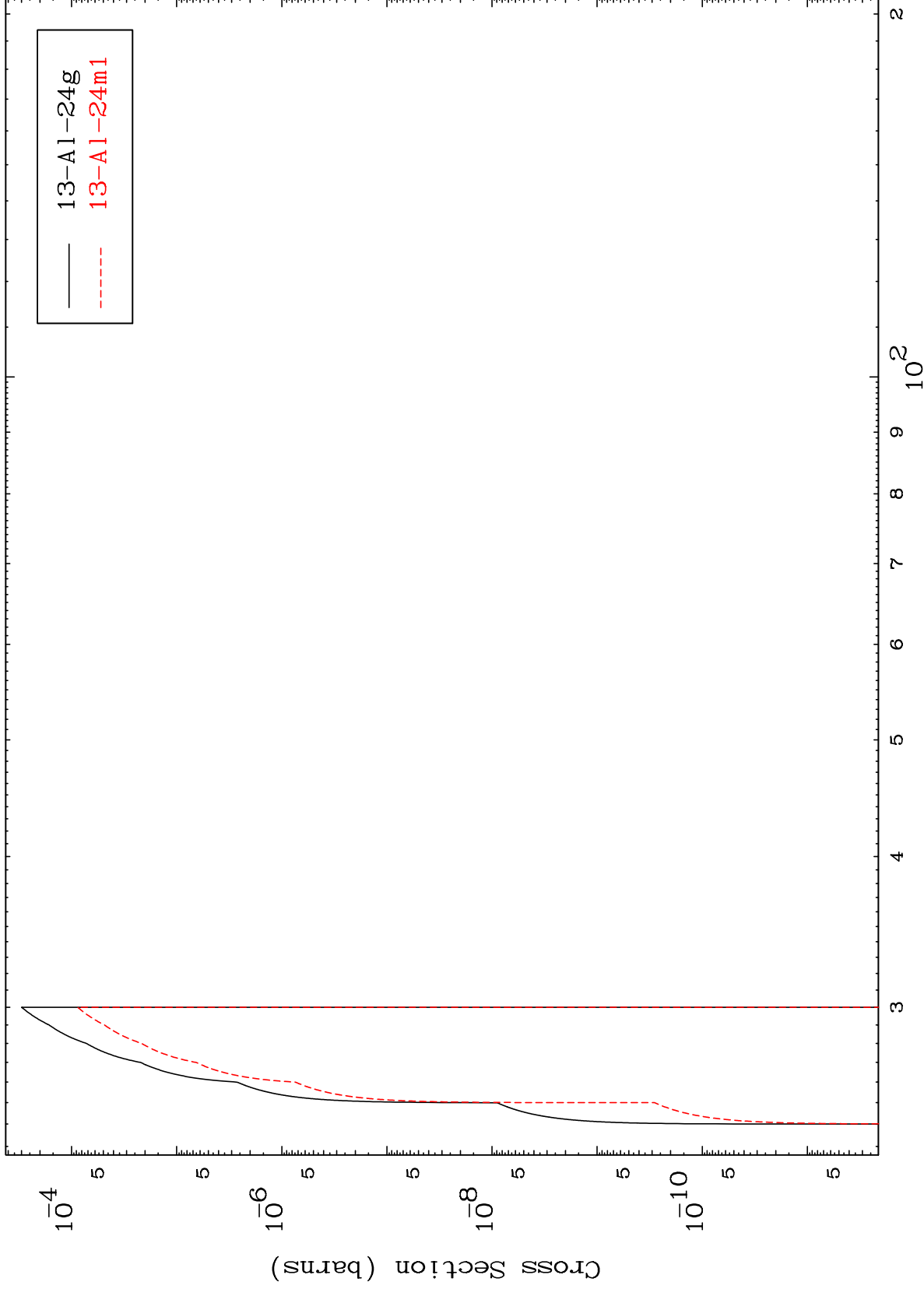
13-Al-24g
13-Al-24m1

MAT 1419

(n,2n) p

14-Si-26

Radionuclide Production Cross Section



24

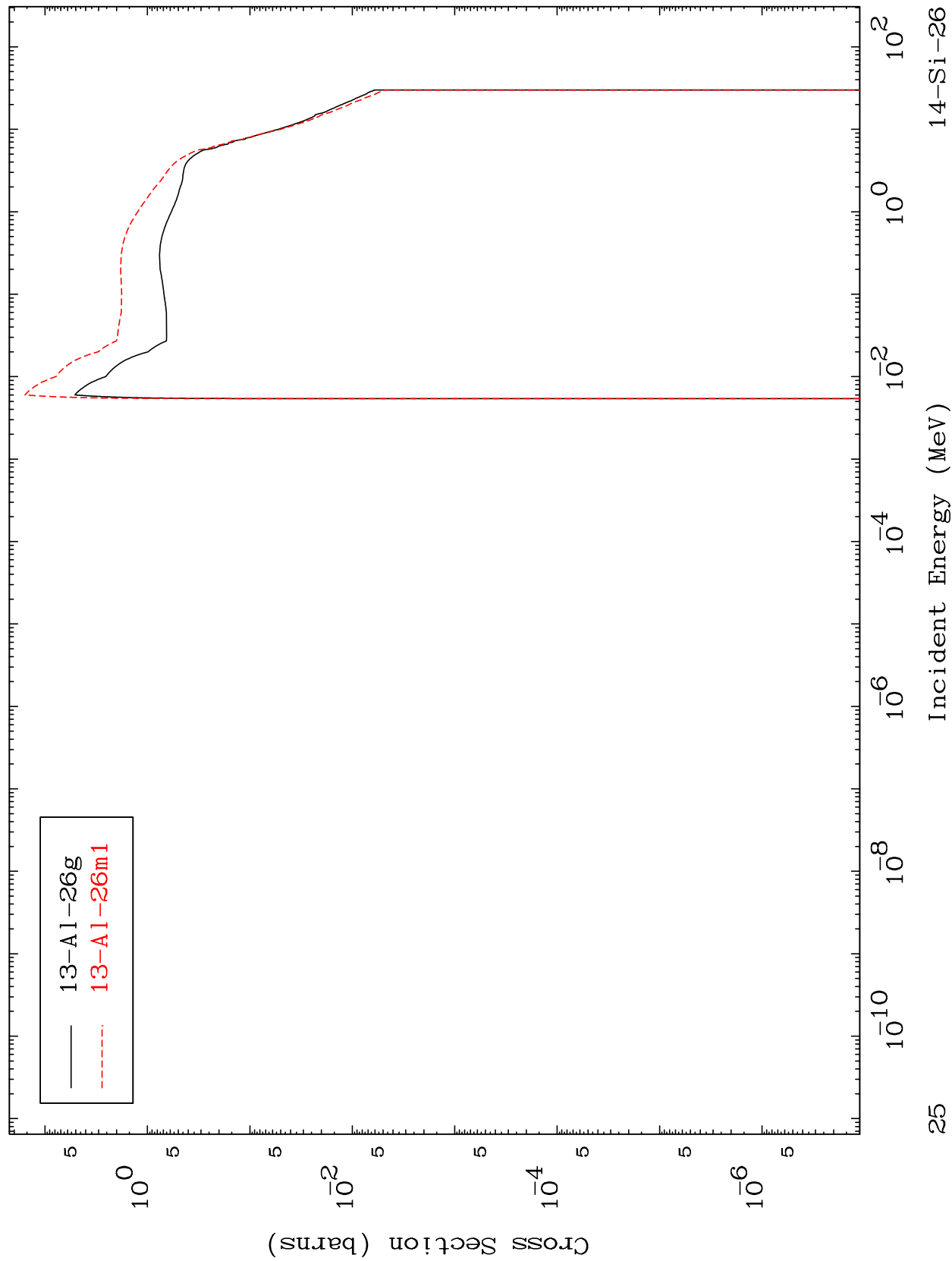
Incident Energy (MeV)

14-Si-26

MAT 1419

14-Si-26

(n,p)
Radionuclide Production Cross Section



25

14-Si-26

(n,t)
Radionuclide Production Cross Section

