

Program EVALPLOT  
(Version 2021-1)

by

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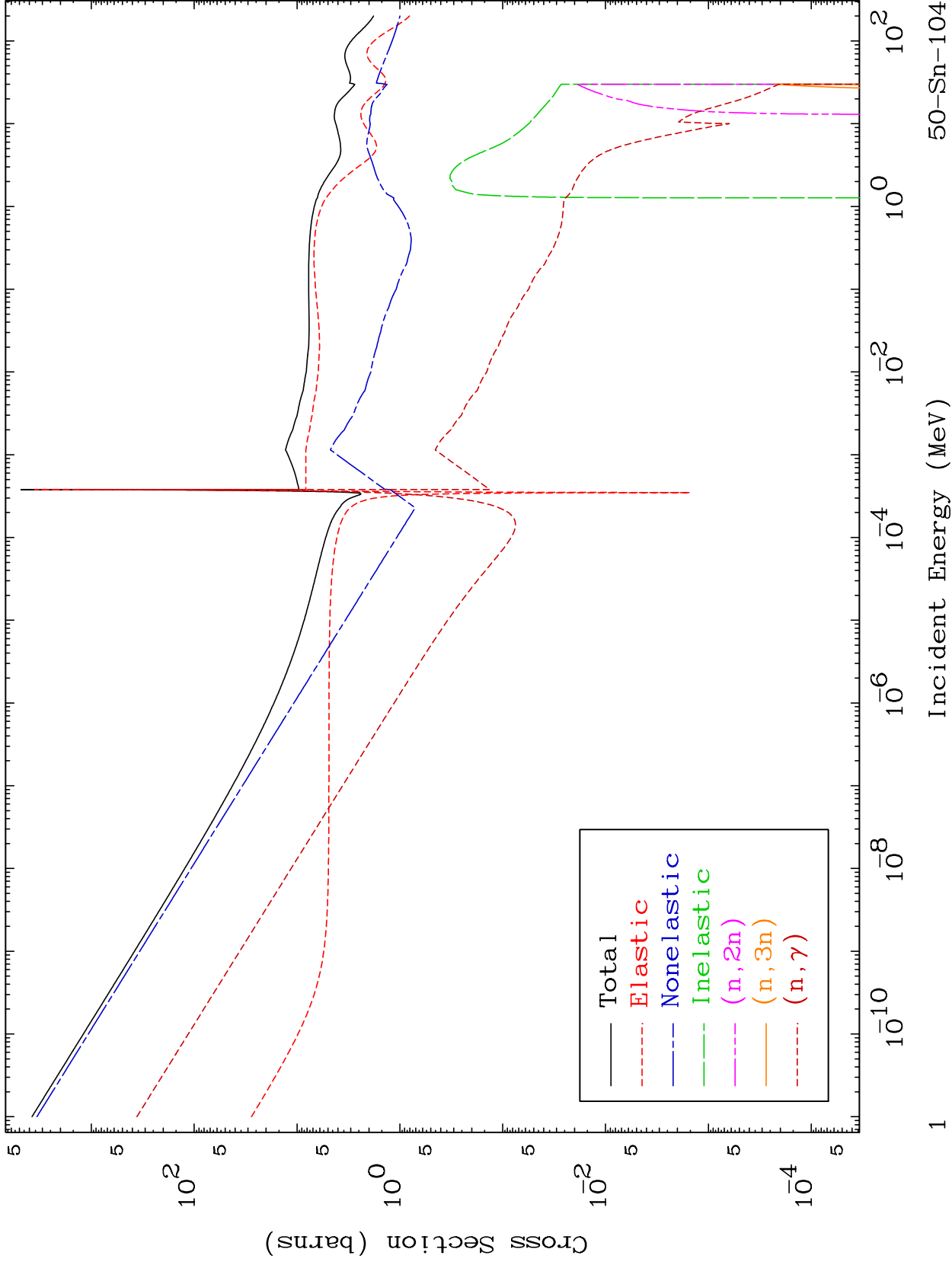
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Press Mouse Button to Start

MAT 5001

Neutron Major  
293 Kelvin Cross Sections

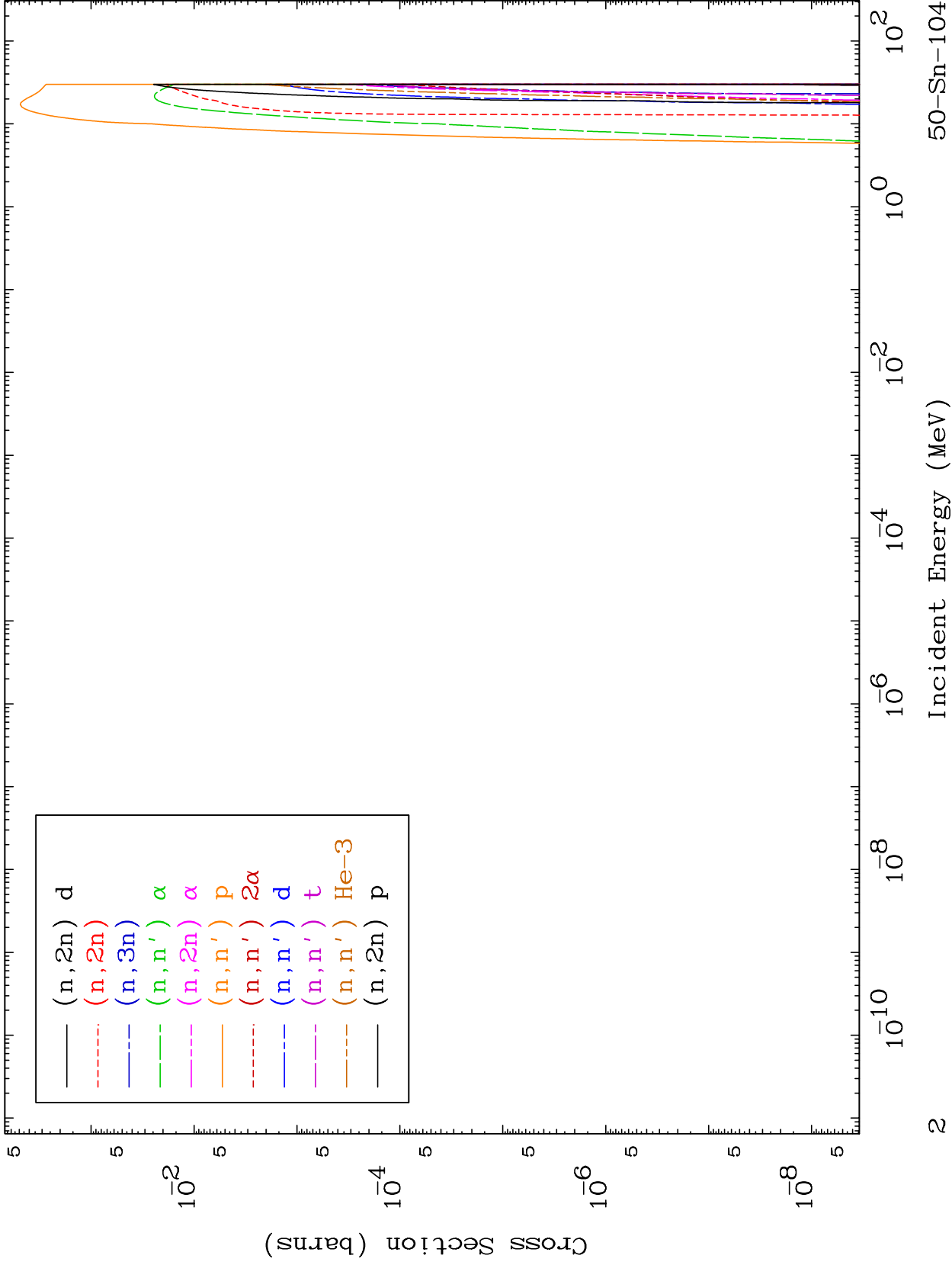
50-Sn-104

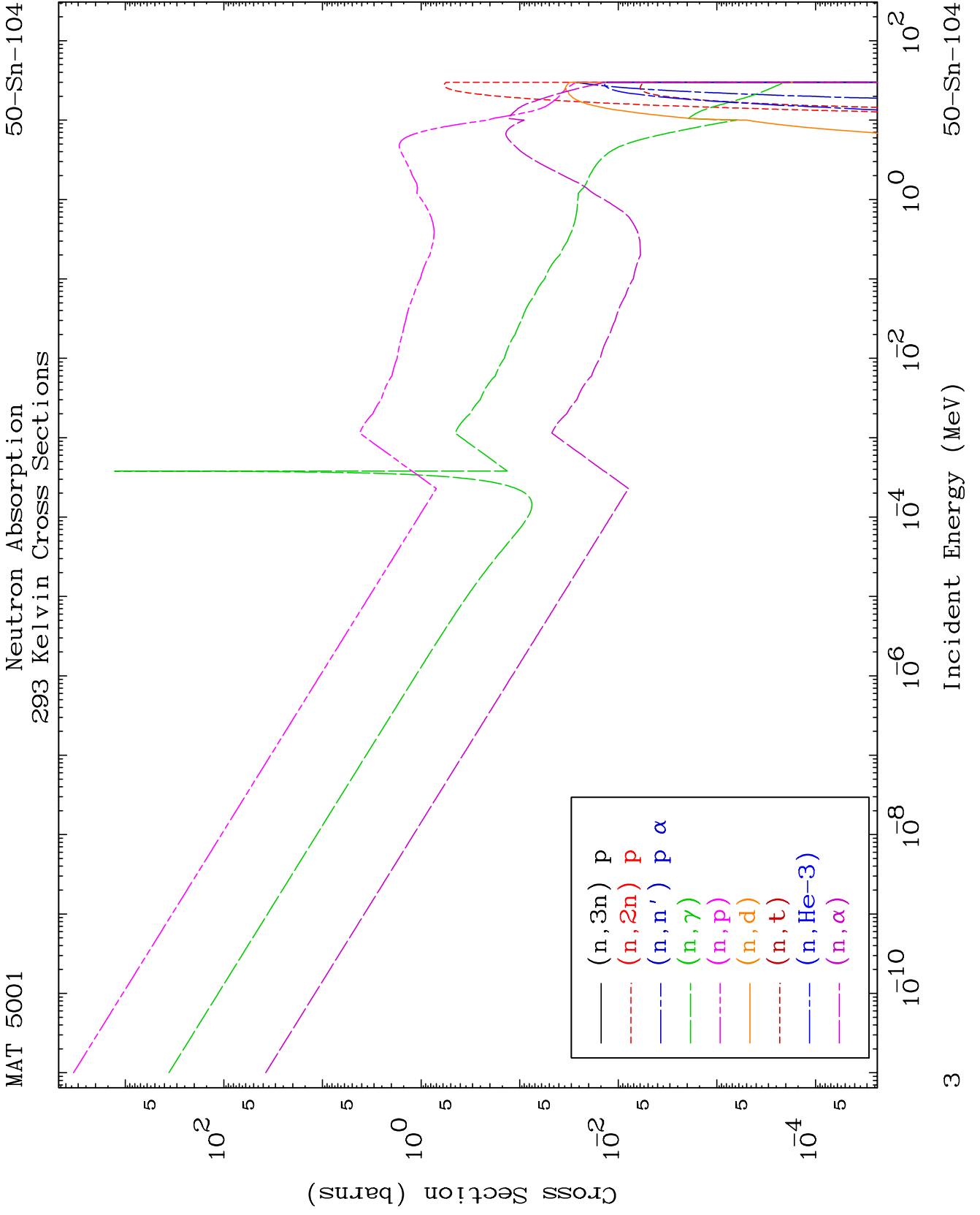


MAT 5001

Neutron Absorption  
293 Kelvin Cross Sections

50-Sn-104



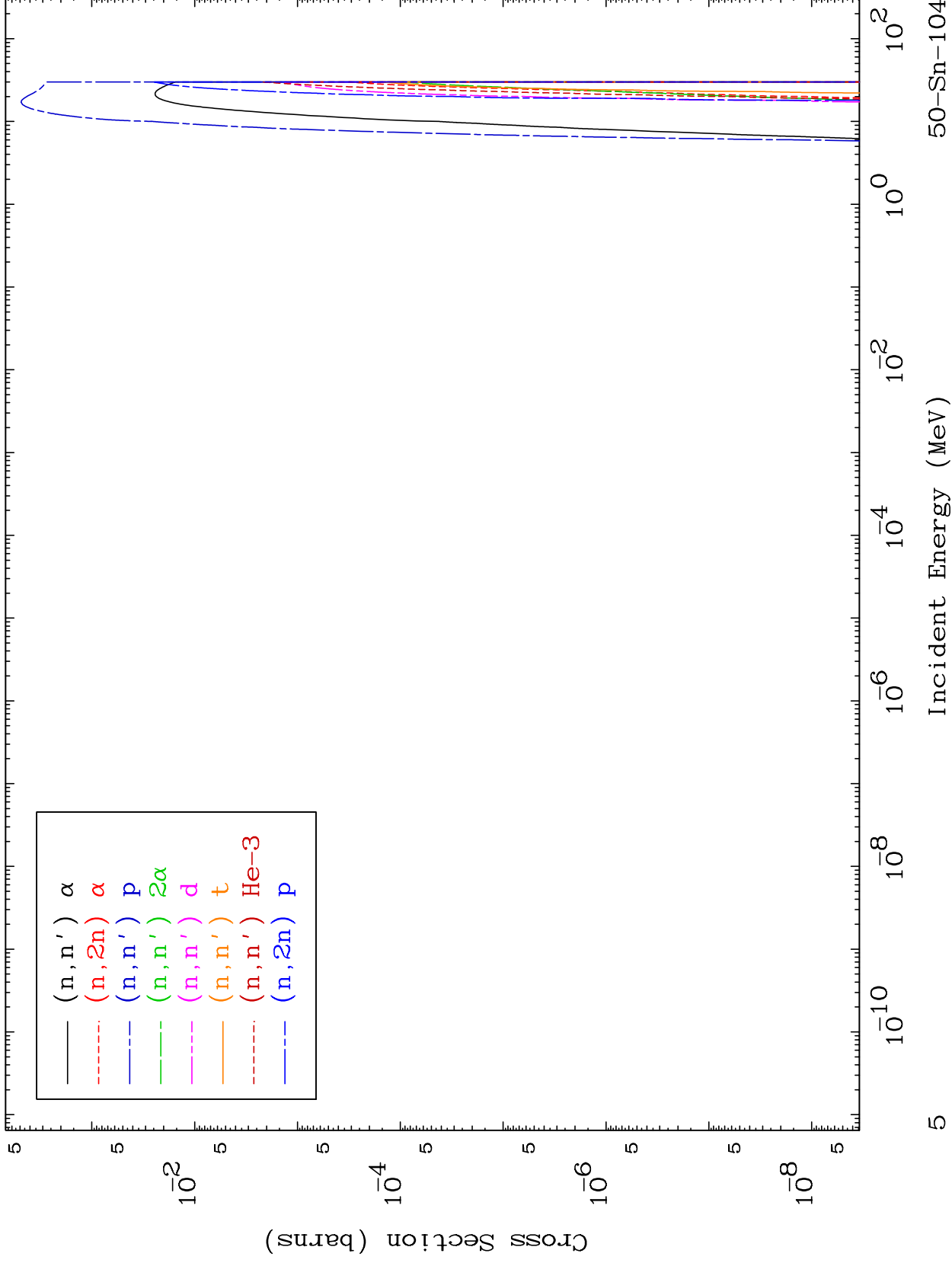




MAT 5001

Charged Particle  
293 Kelvin Cross Sections

50-Sn-104

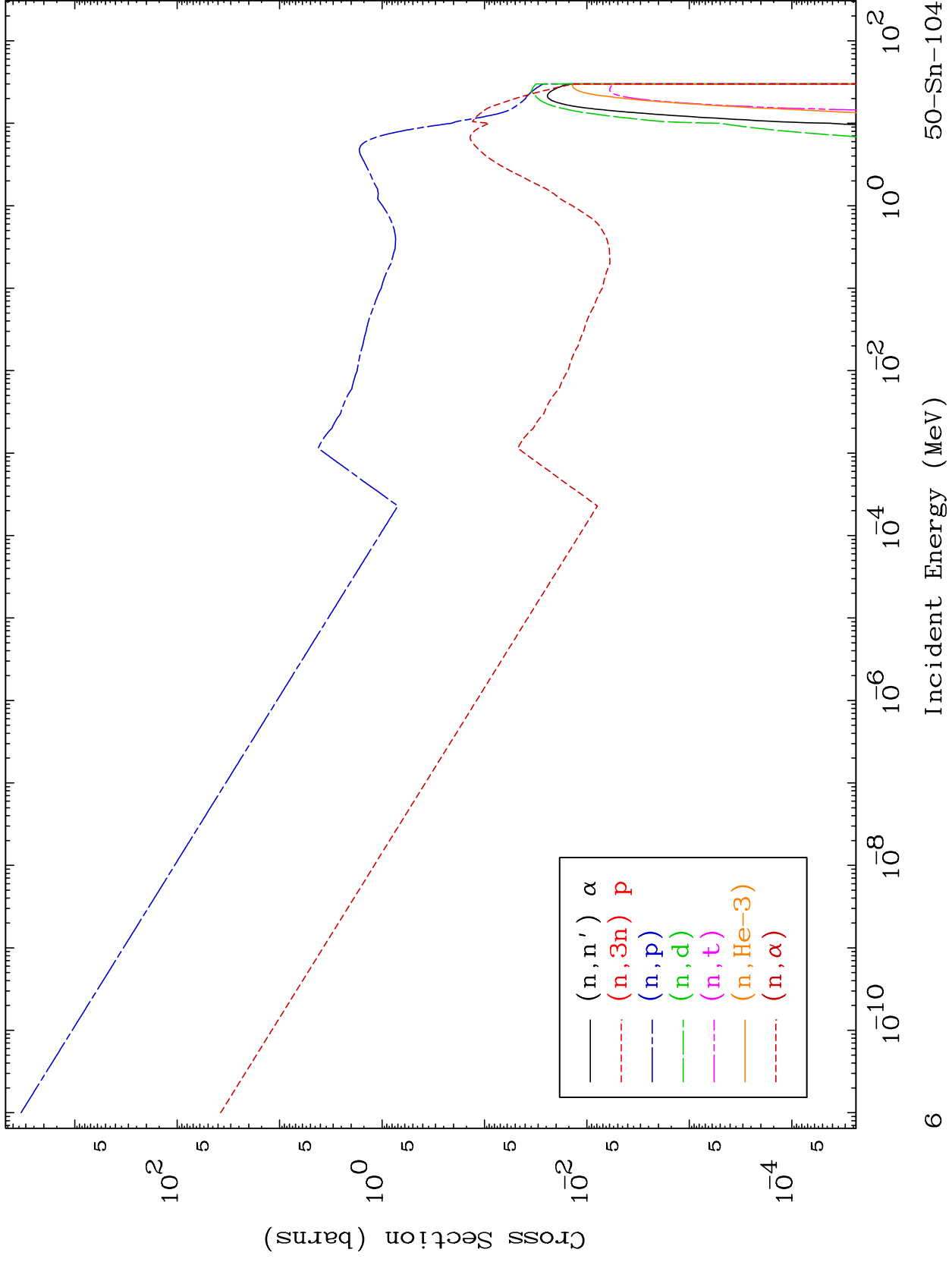


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MAT 5001

Charged Particle  
293 Kelvin Cross Sections

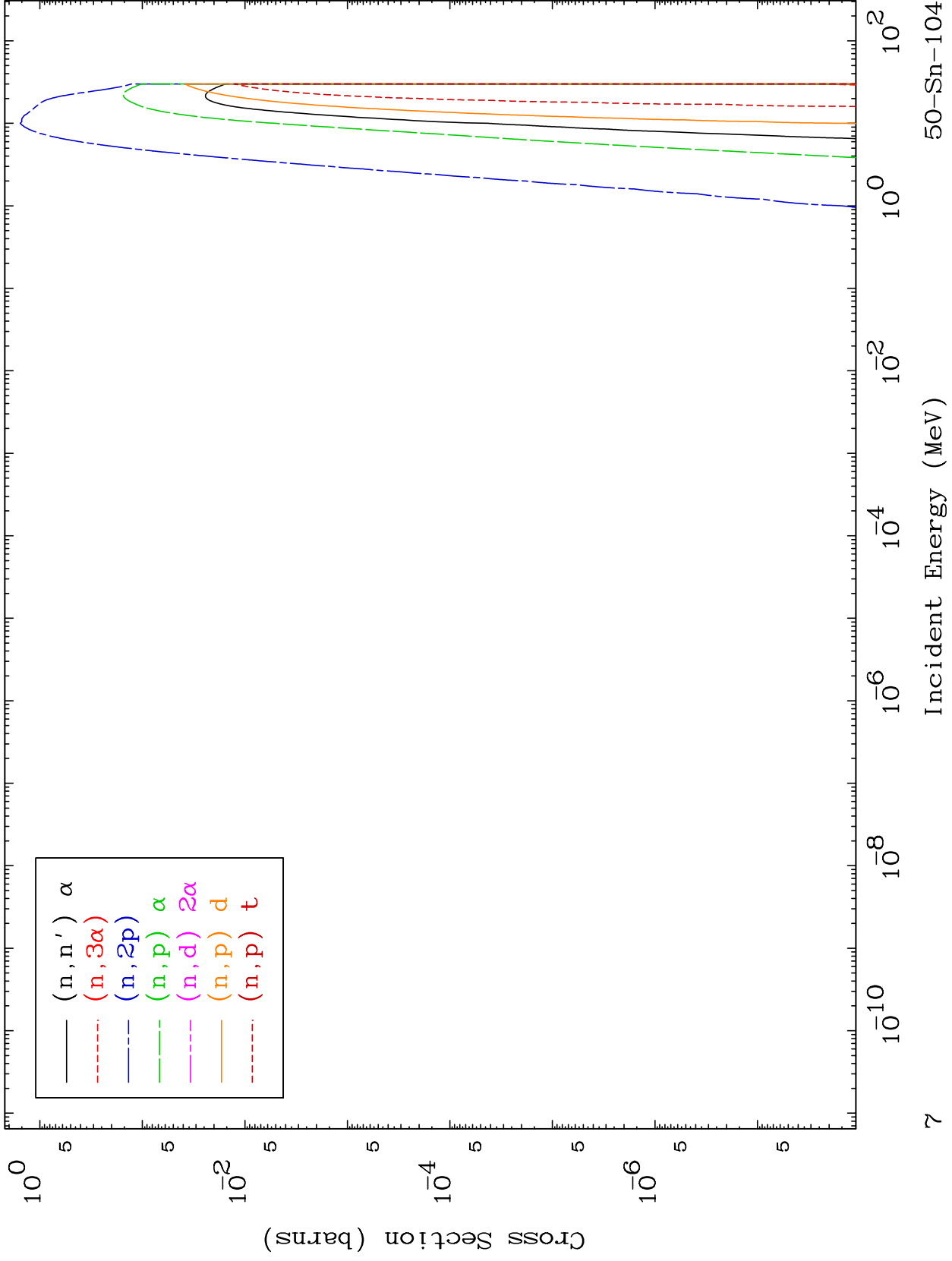
50-Sn-104



MAT 5001

Charged Particle  
293 Kelvin Cross Sections

50-Sn-104

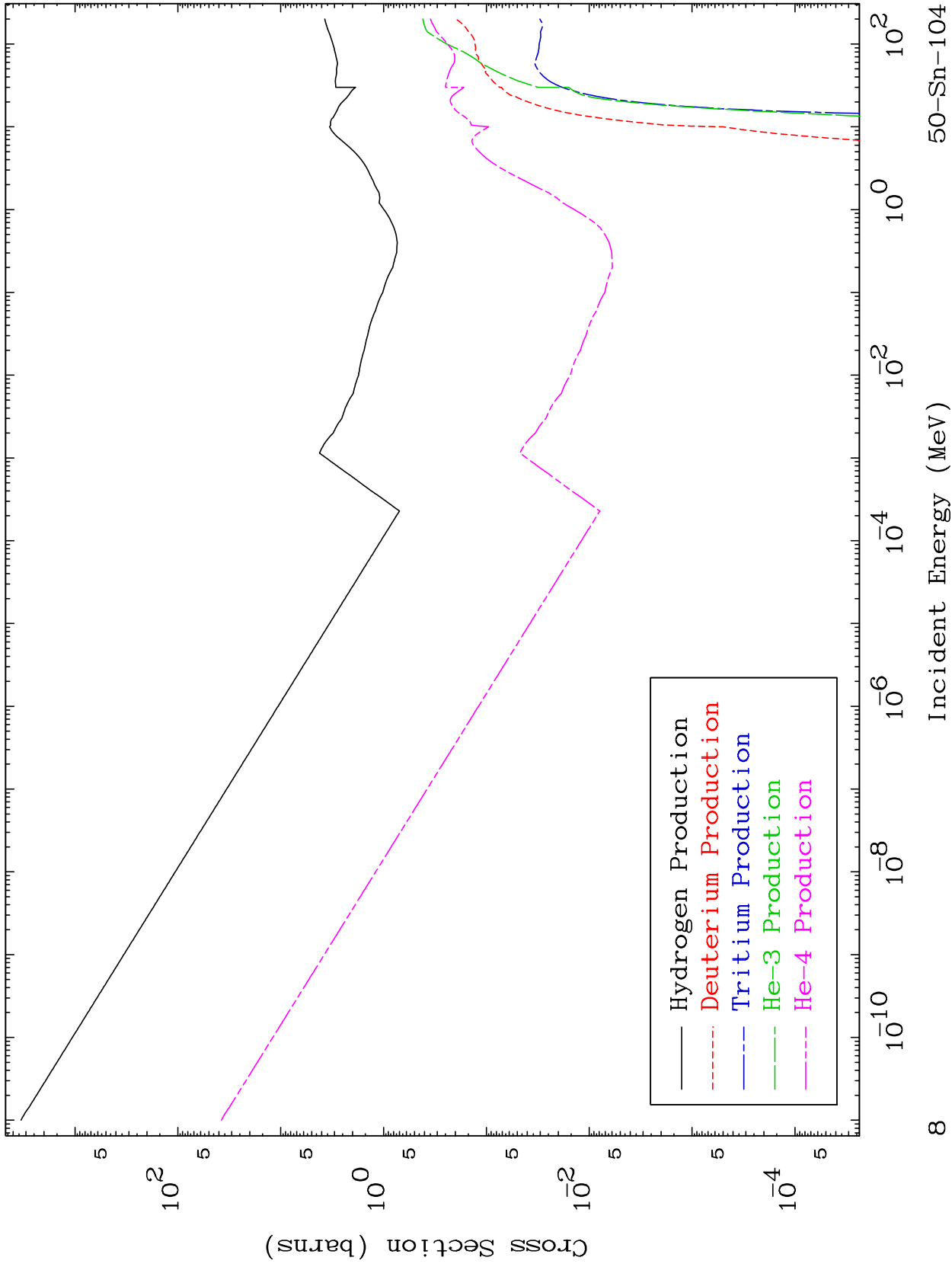


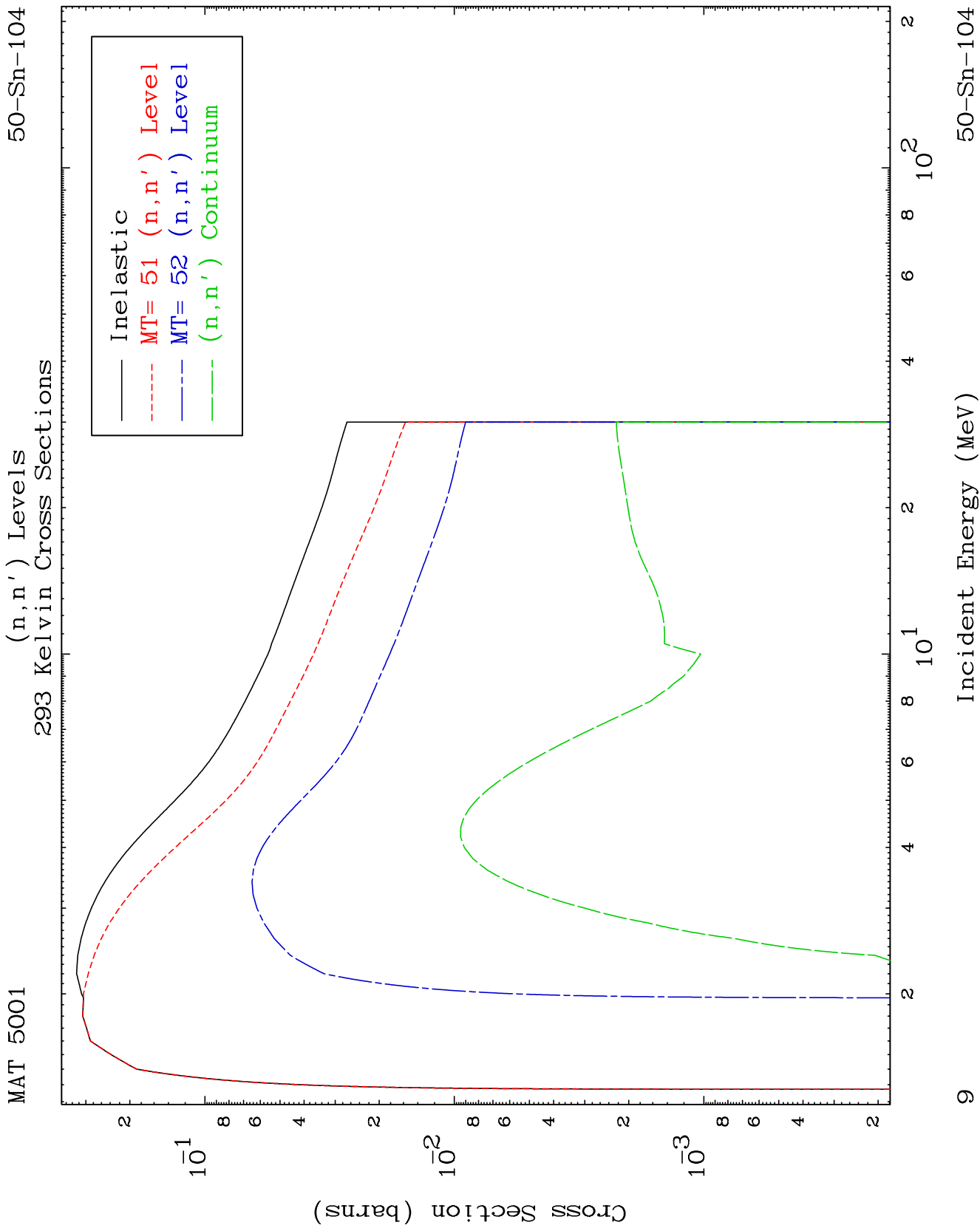


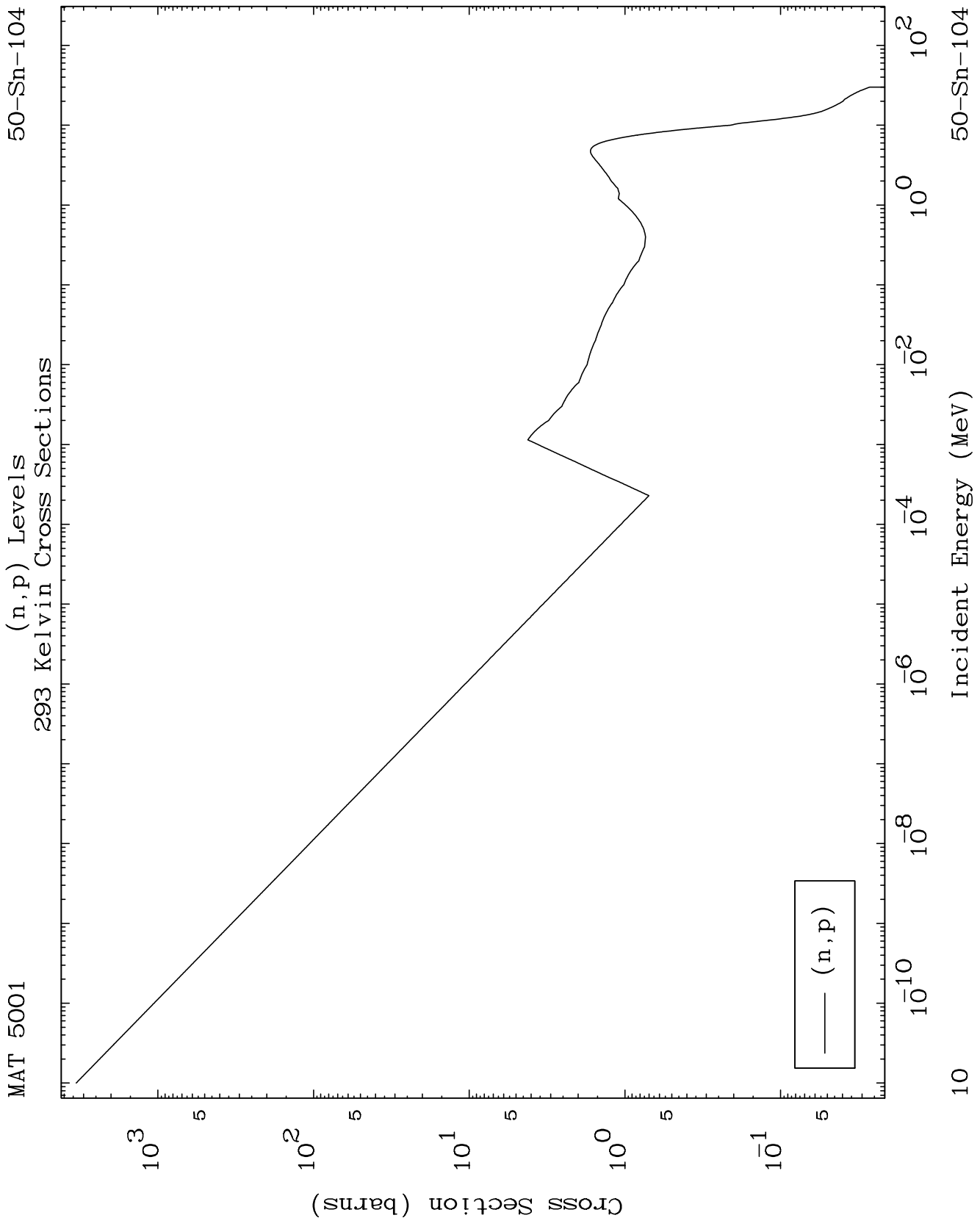
MAT 5001

Particle Production  
293 Kelvin Cross Sections

50-Sn-104



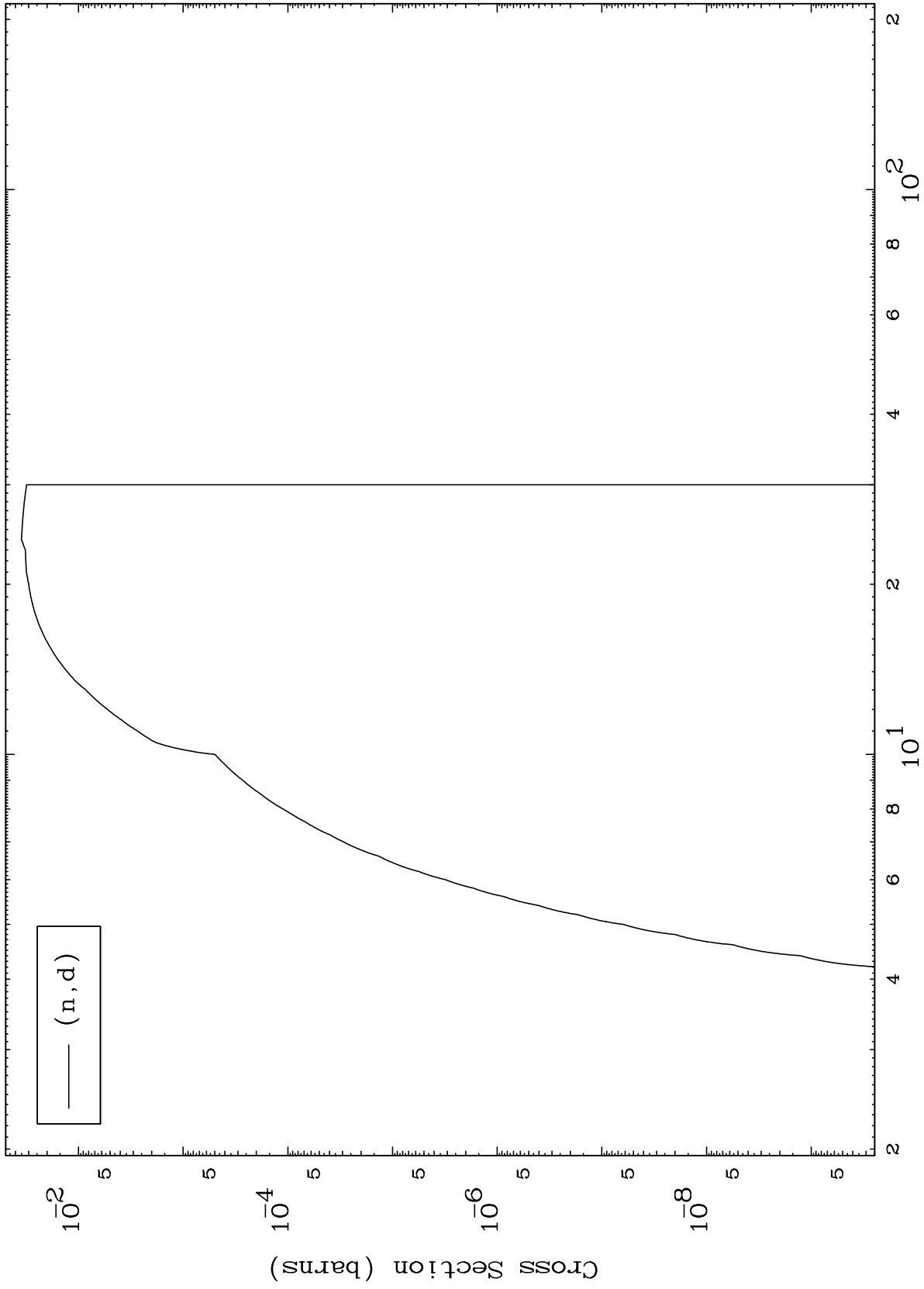




MAT 5001

(n,d) Levels  
293 Kelvin Cross Sections

50-Sn-104



11

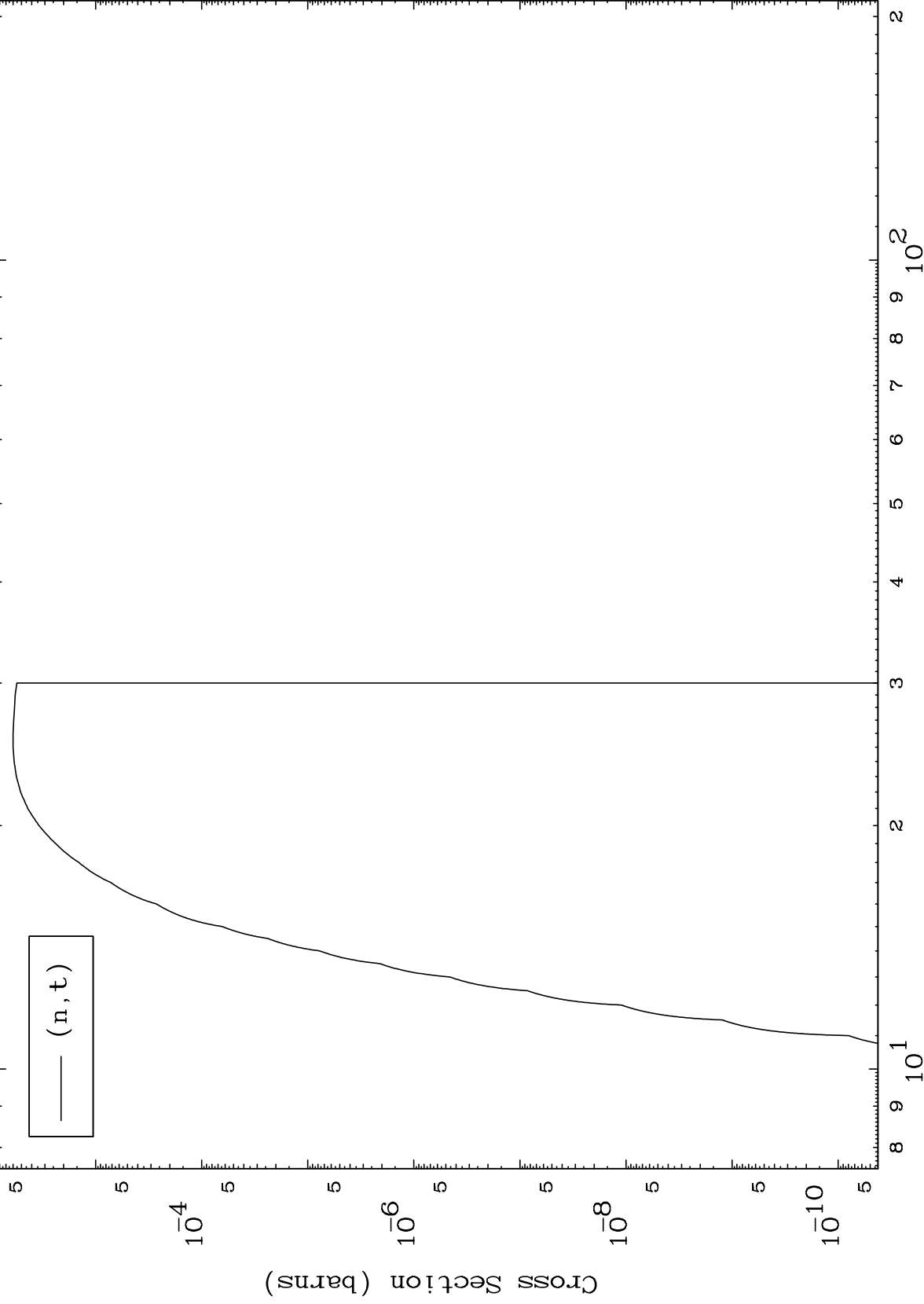
Incident Energy (MeV)

50-Sn-104

MAT 5001

(n,t) Levels  
293 Kelvin Cross Sections

50-Sn-104



12

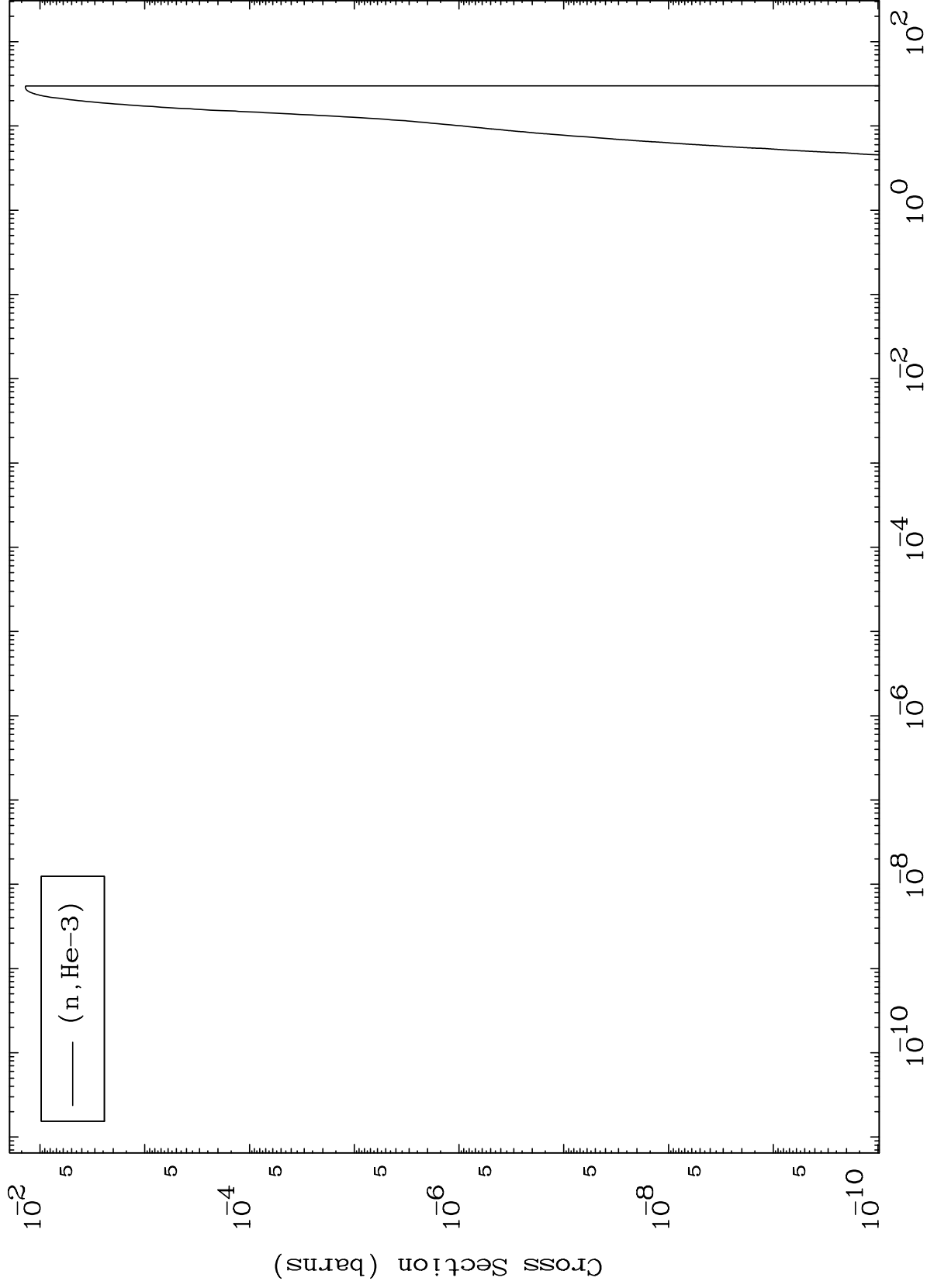
Incident Energy (MeV)

50-Sn-104

MAT 5001

(n,He3) Levels  
293 Kelvin Cross Sections

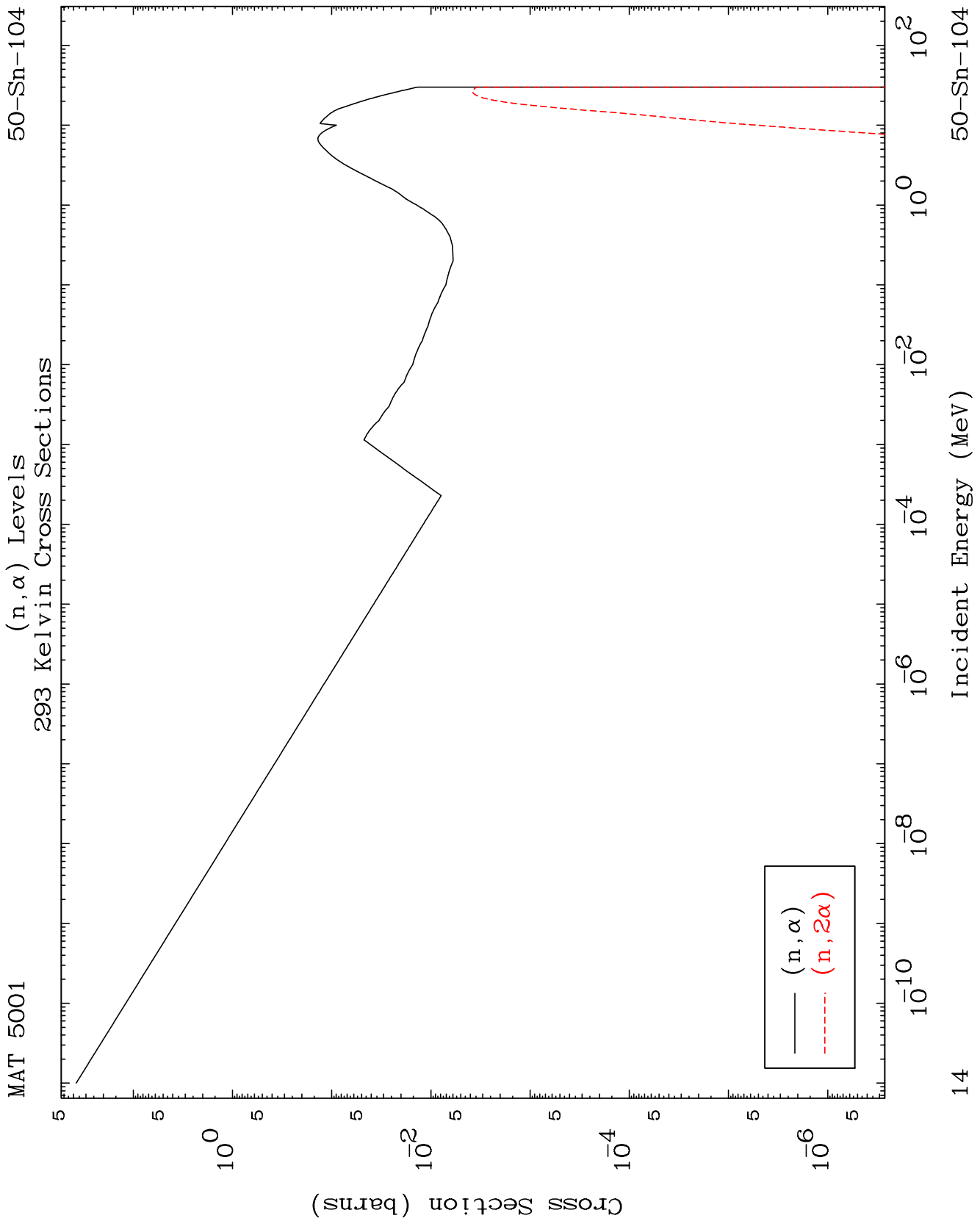
50-Sn-104



13

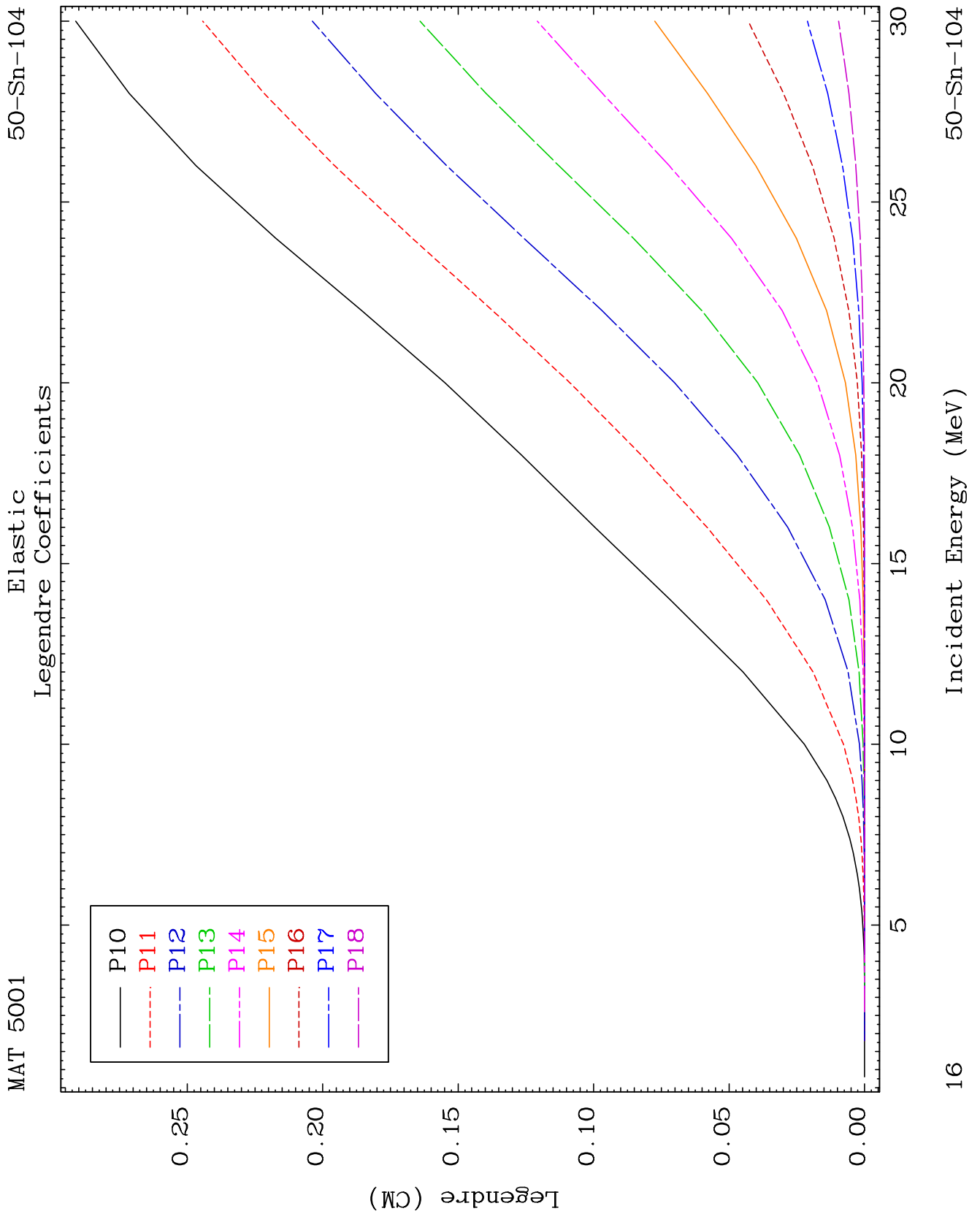
Incident Energy (MeV)

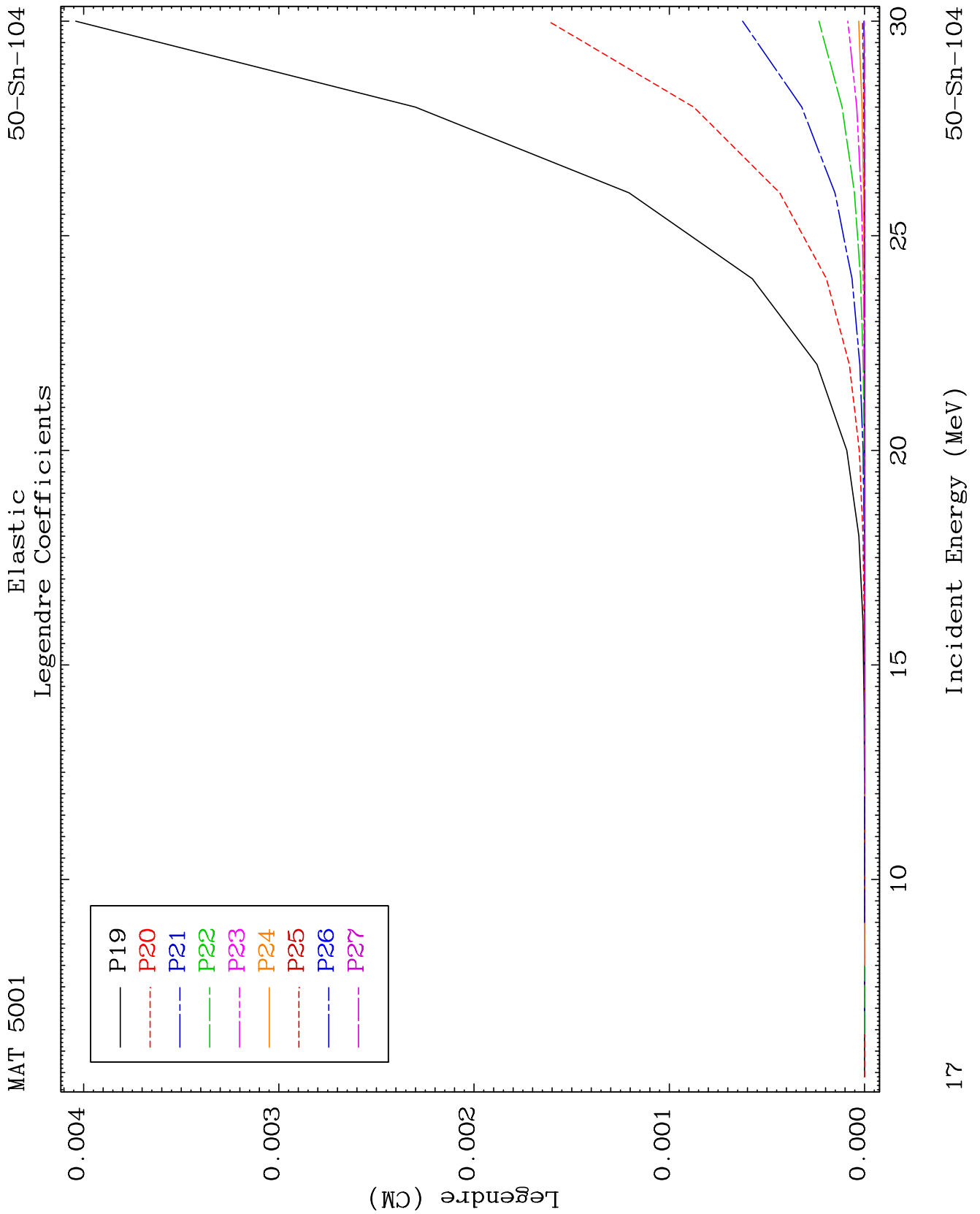
50-Sn-104

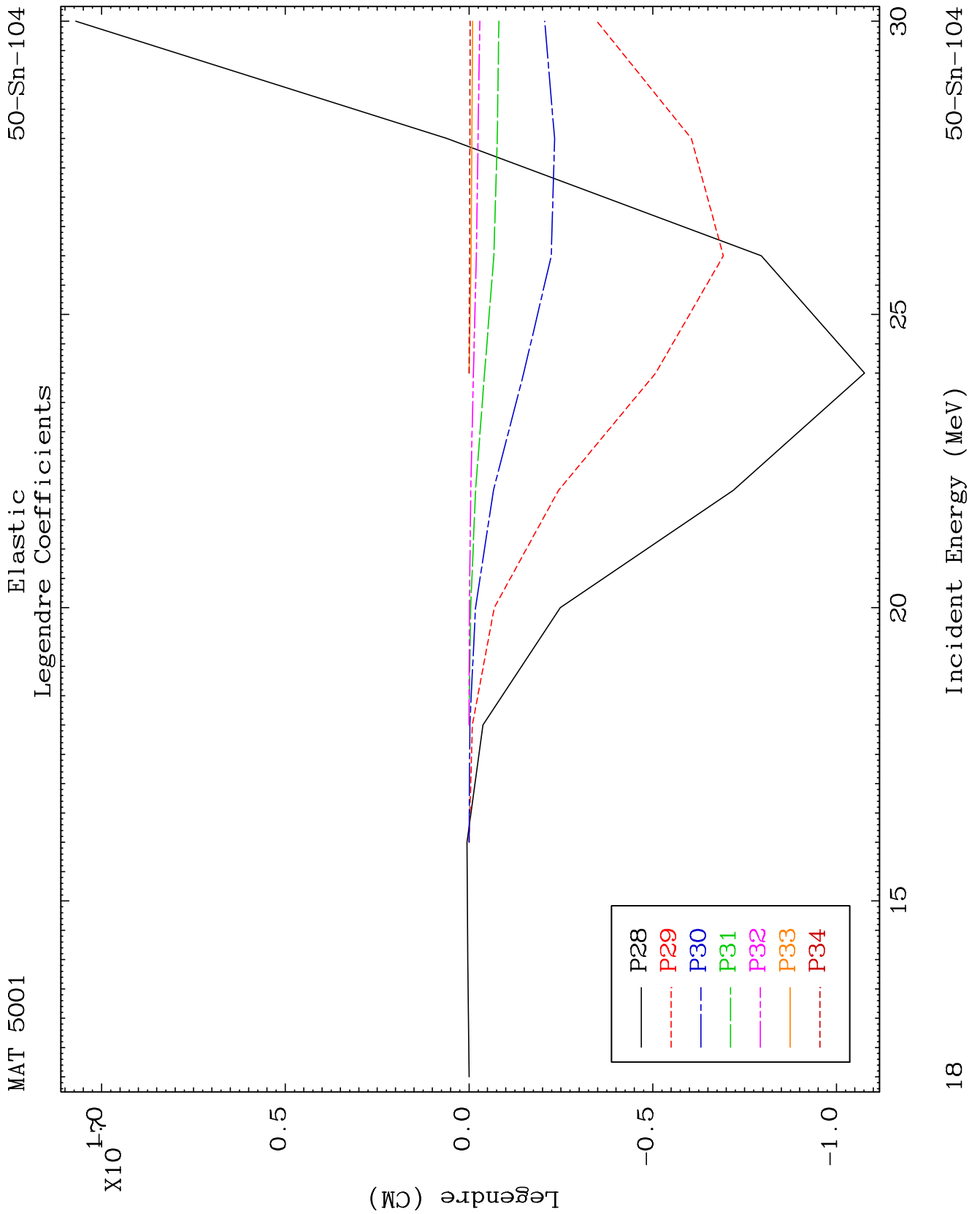


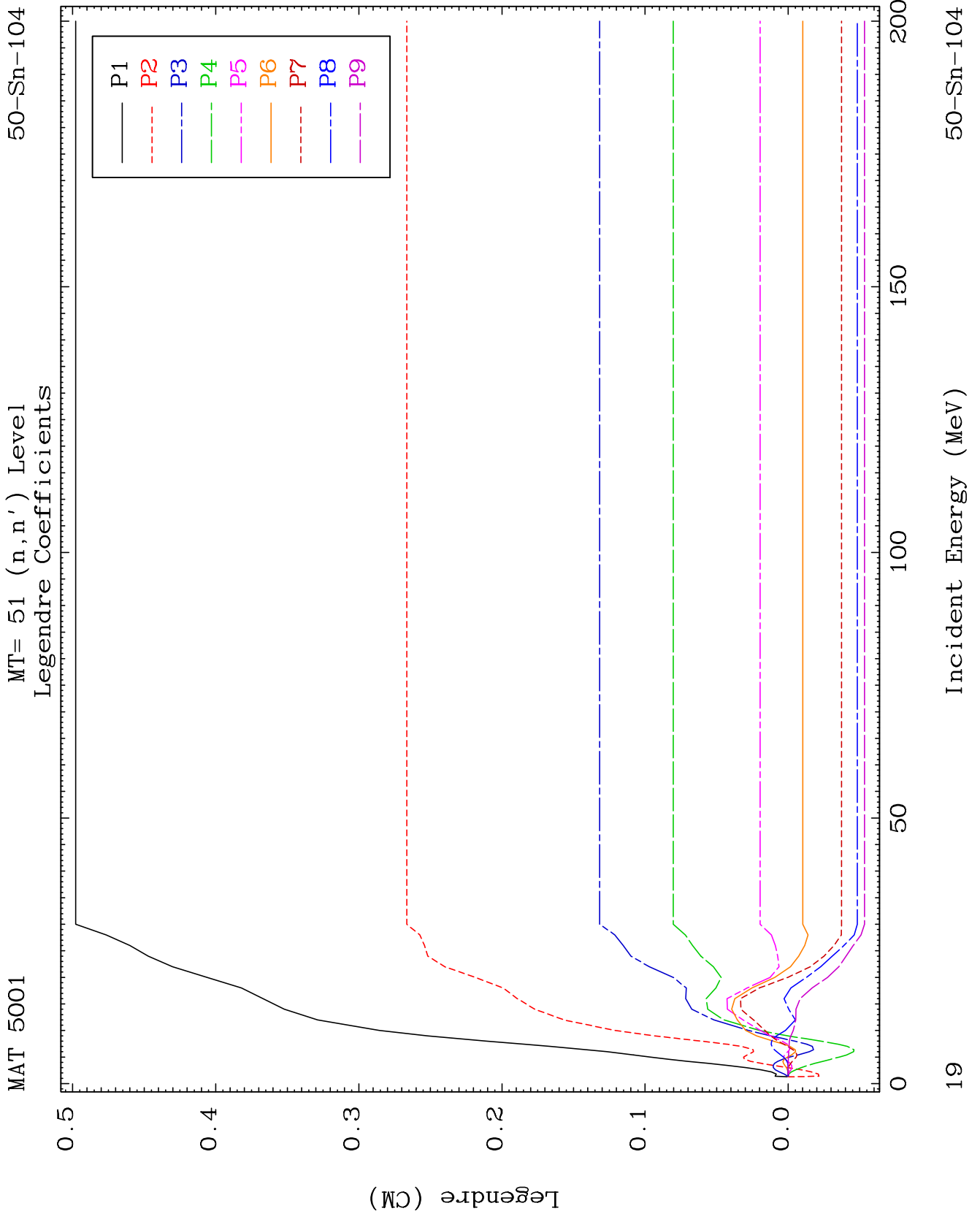


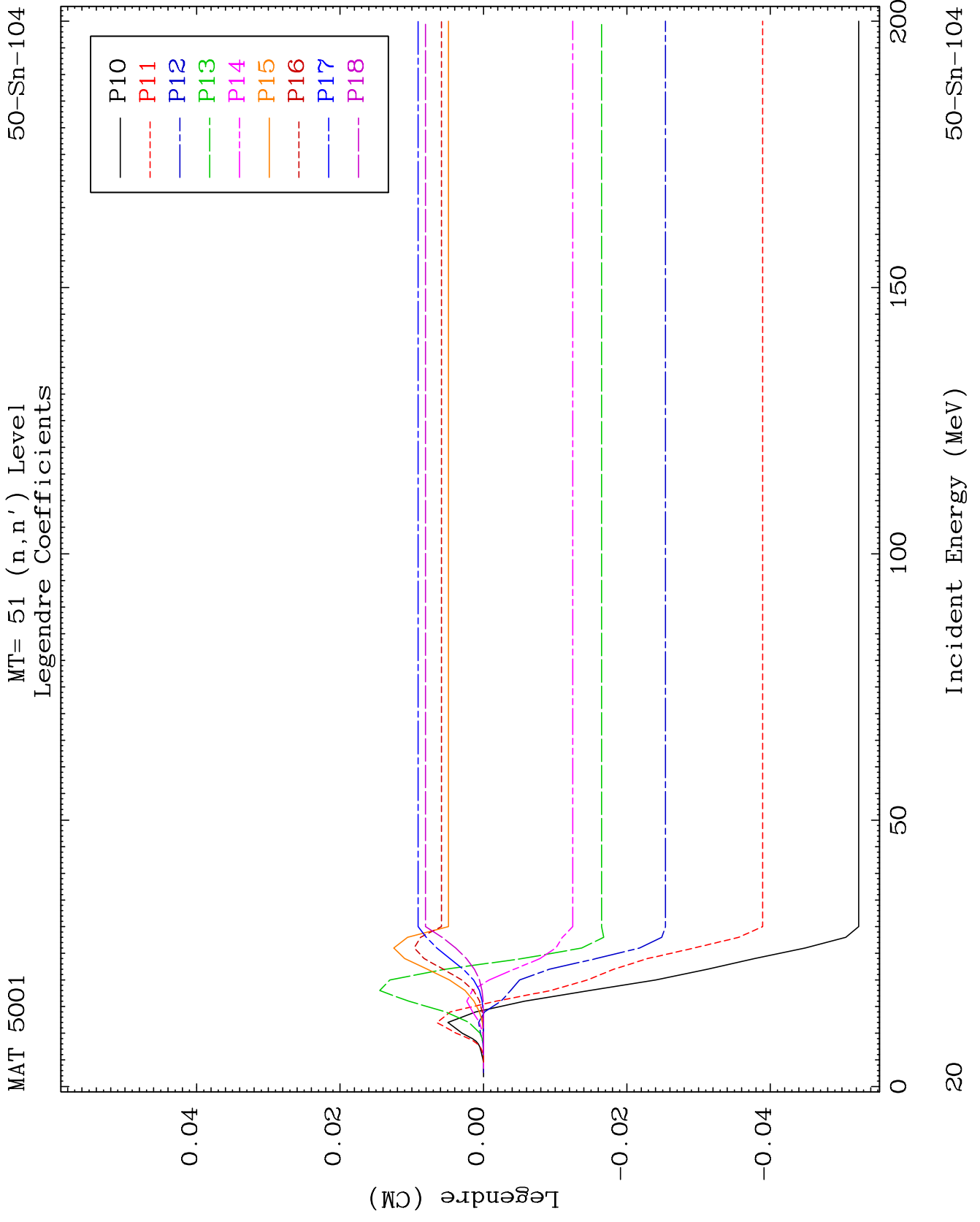


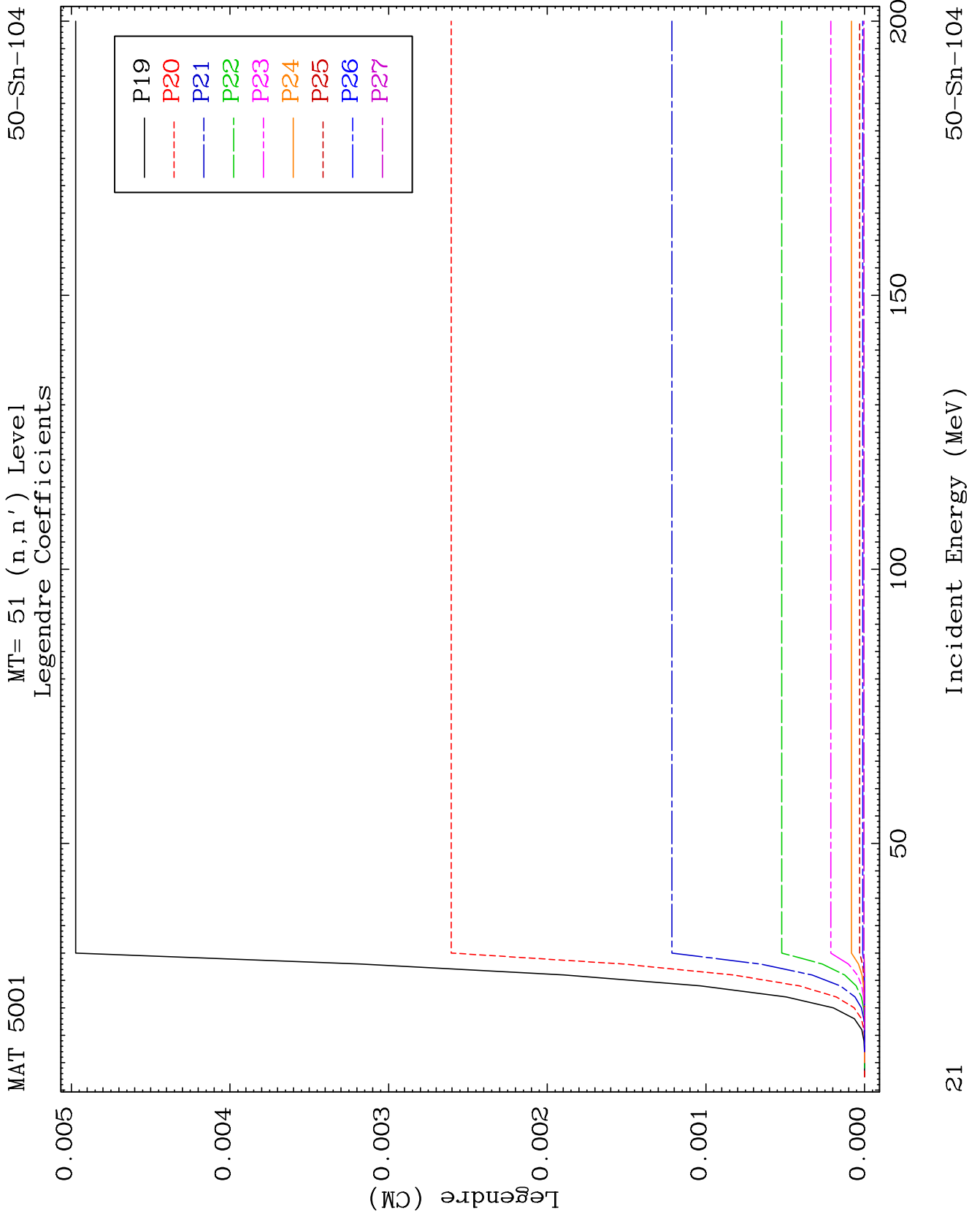


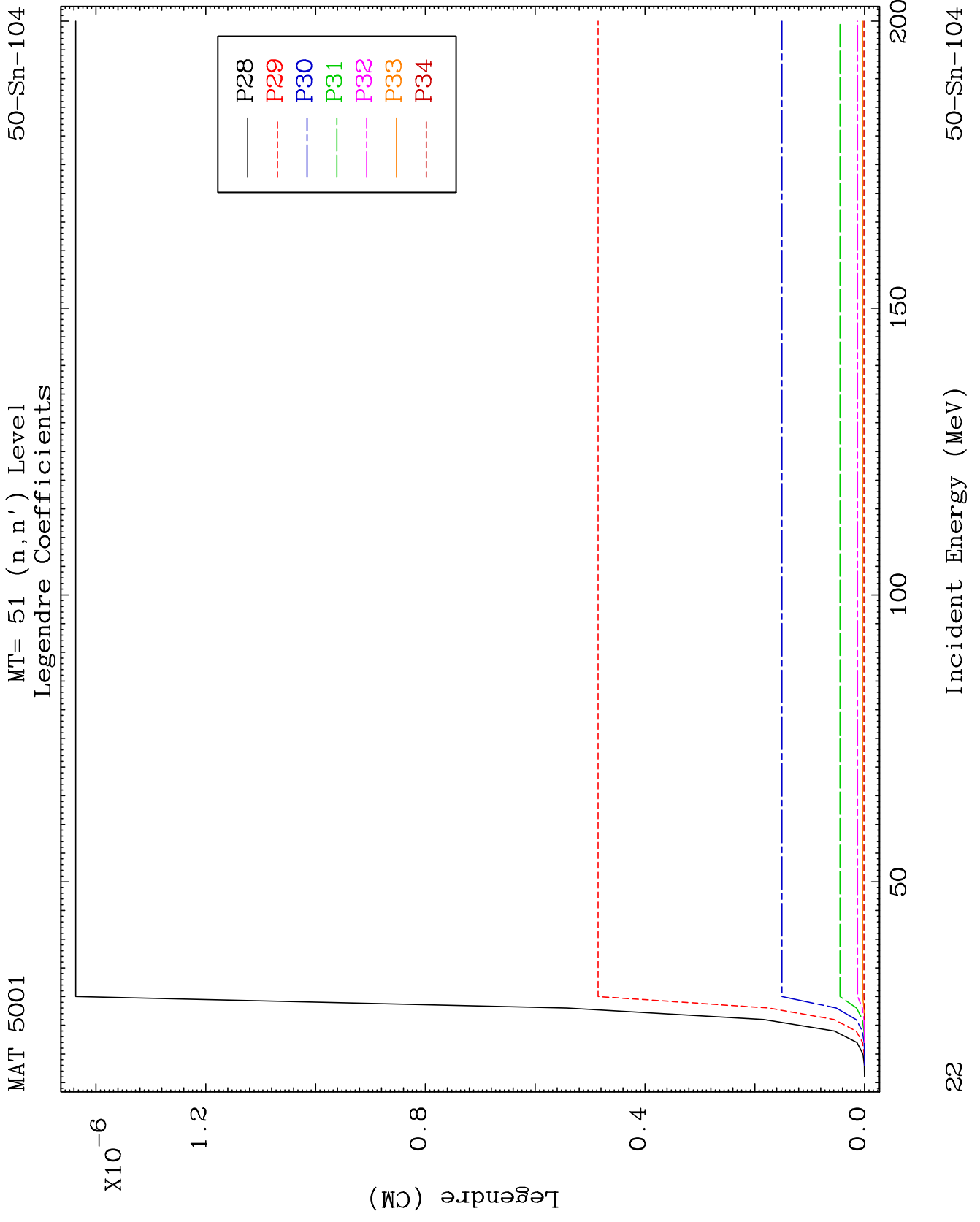


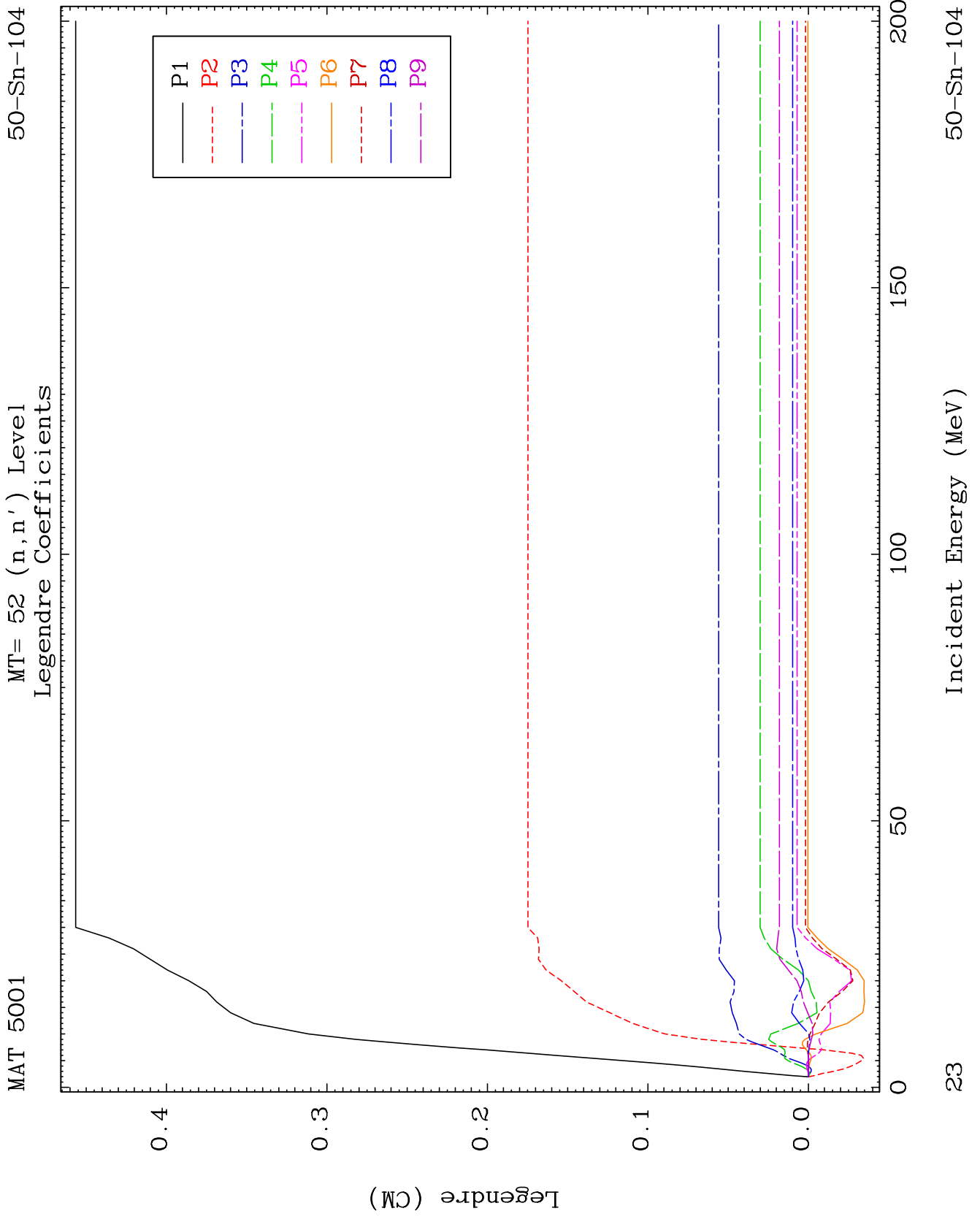






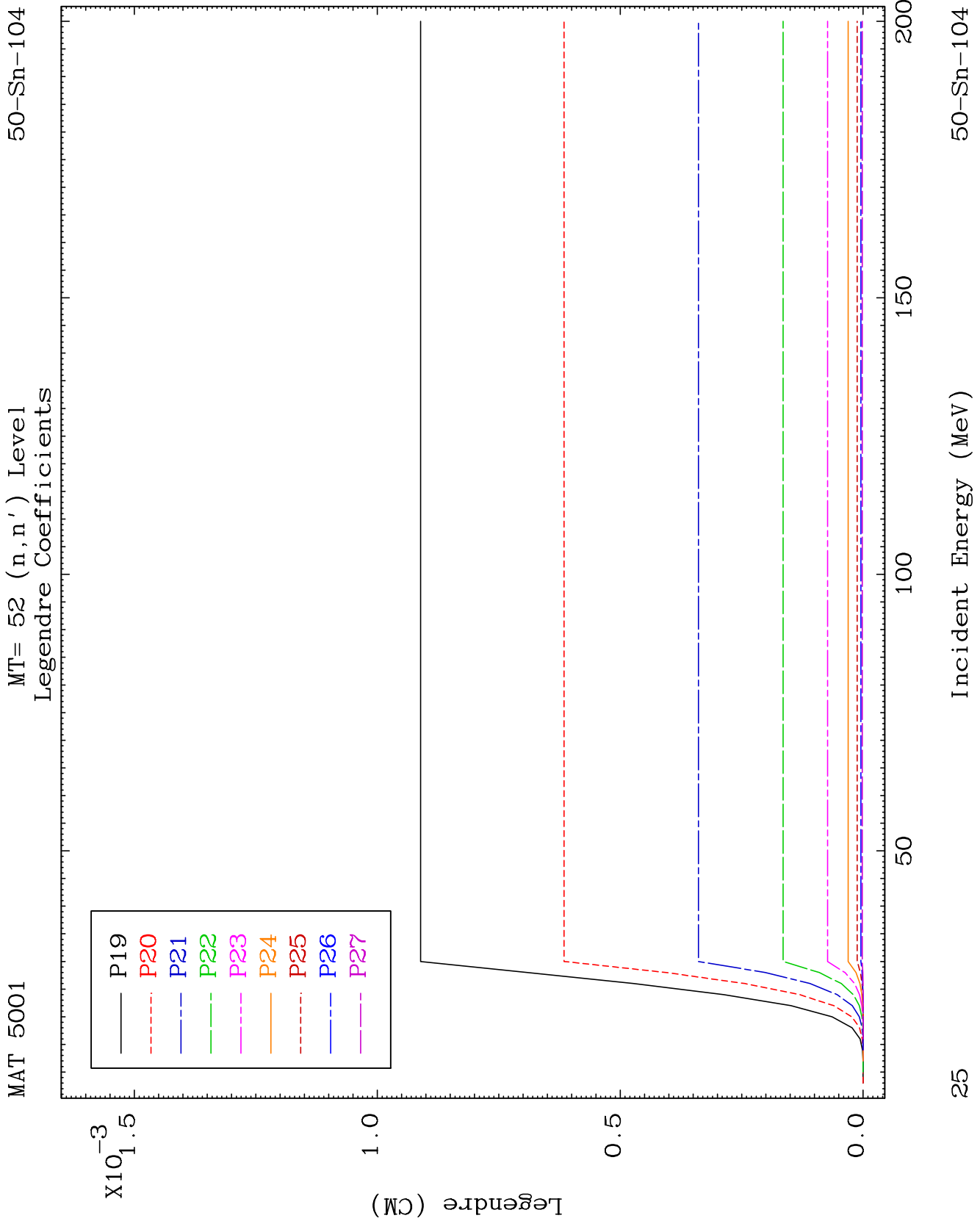


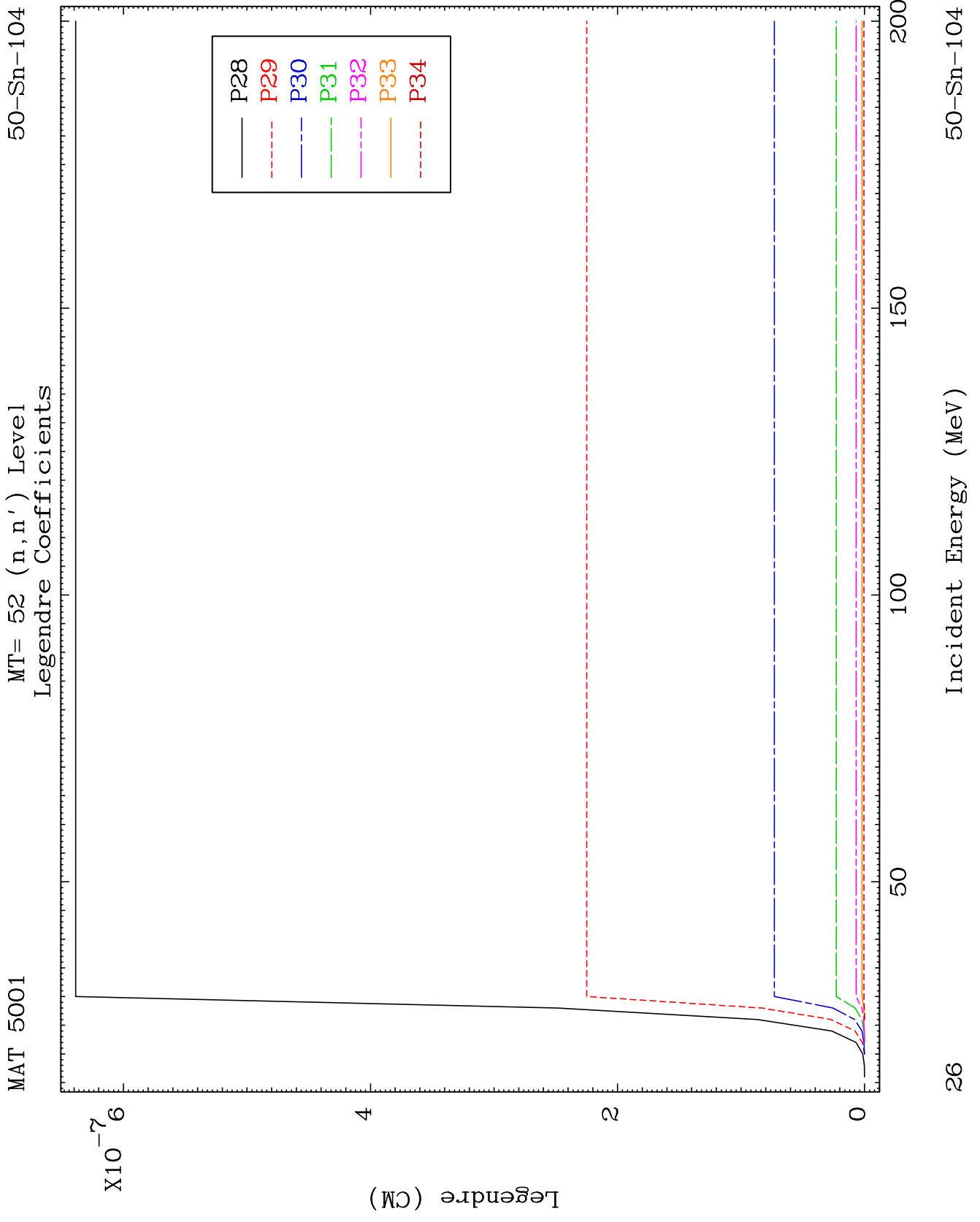










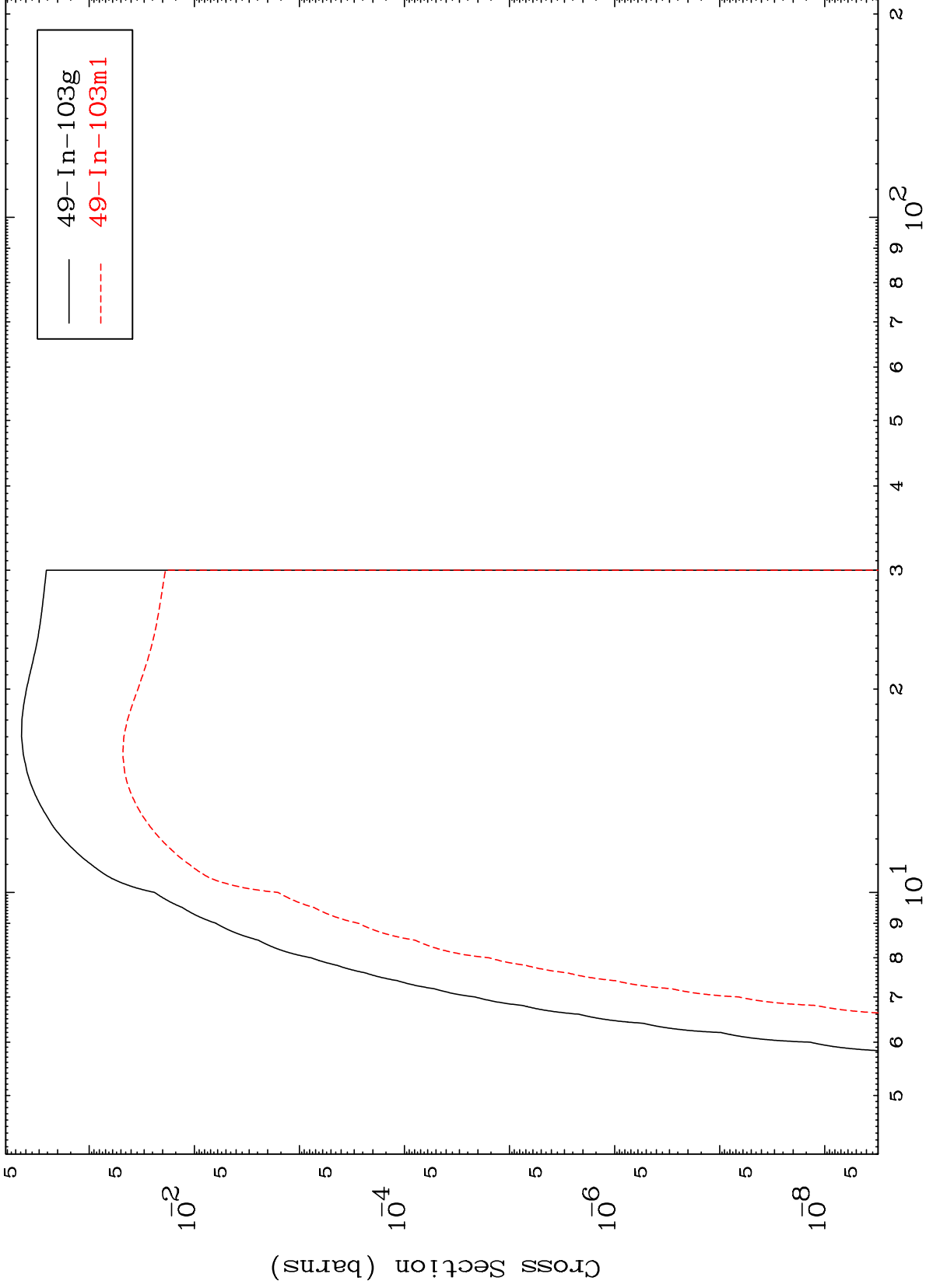


MAT 5001

(n,n') p

50-Sn-104

Radionuclide Production Cross Section



27

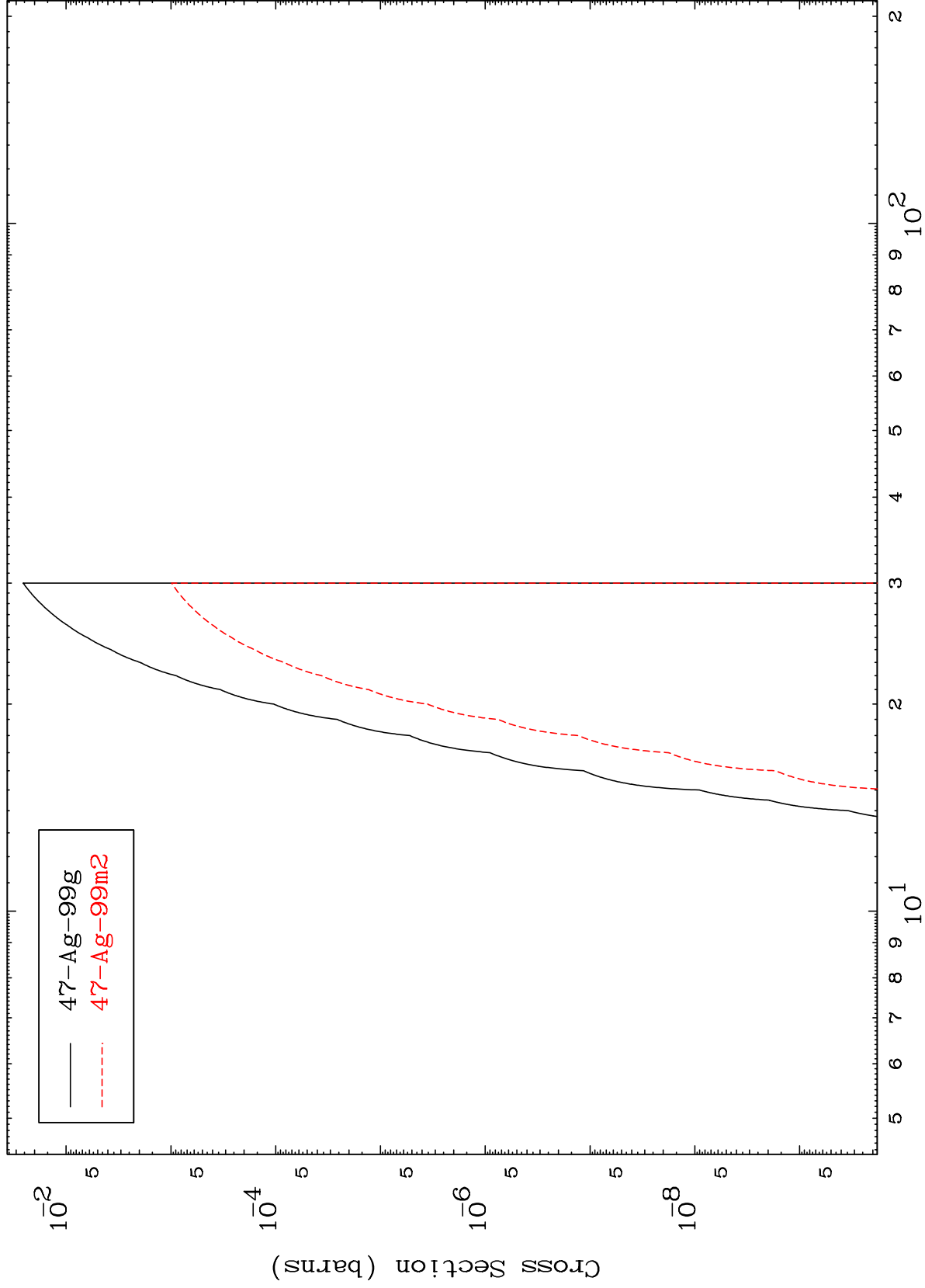
Incident Energy (MeV)

50-Sn-104

MAT 5001

50-Sn-104

(n,n') p  $\alpha$   
Radionuclide Production Cross Section



28

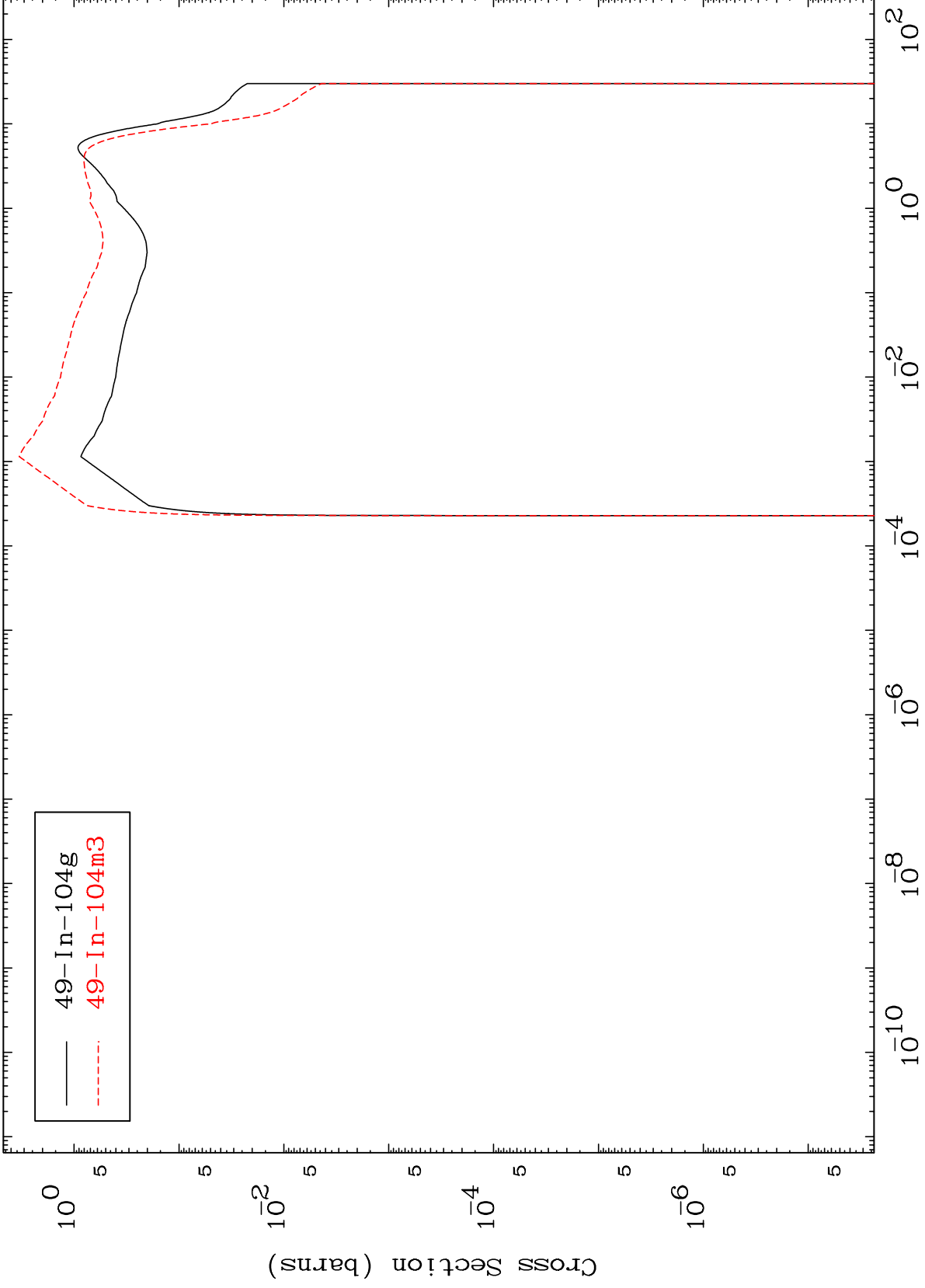
Incident Energy (MeV)

50-Sn-104

MAT 5001

50-Sn-104

(n,p)  
Radionuclide Production Cross Section

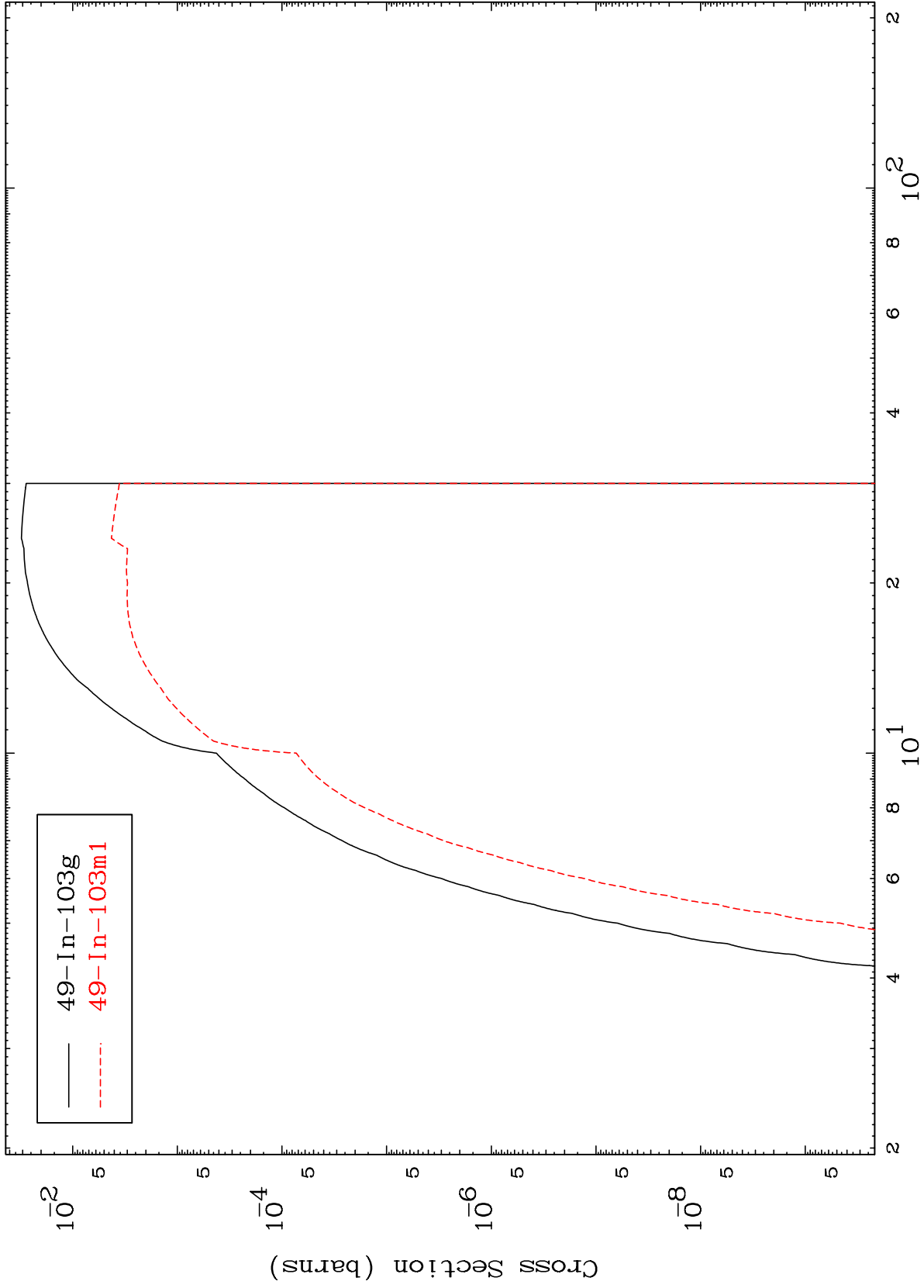


— 49-In-104g  
- - - 49-In-104m3

MAT 5001

50-Sn-104

(n,d)  
Radionuclide Production Cross Section



50-Sn-104

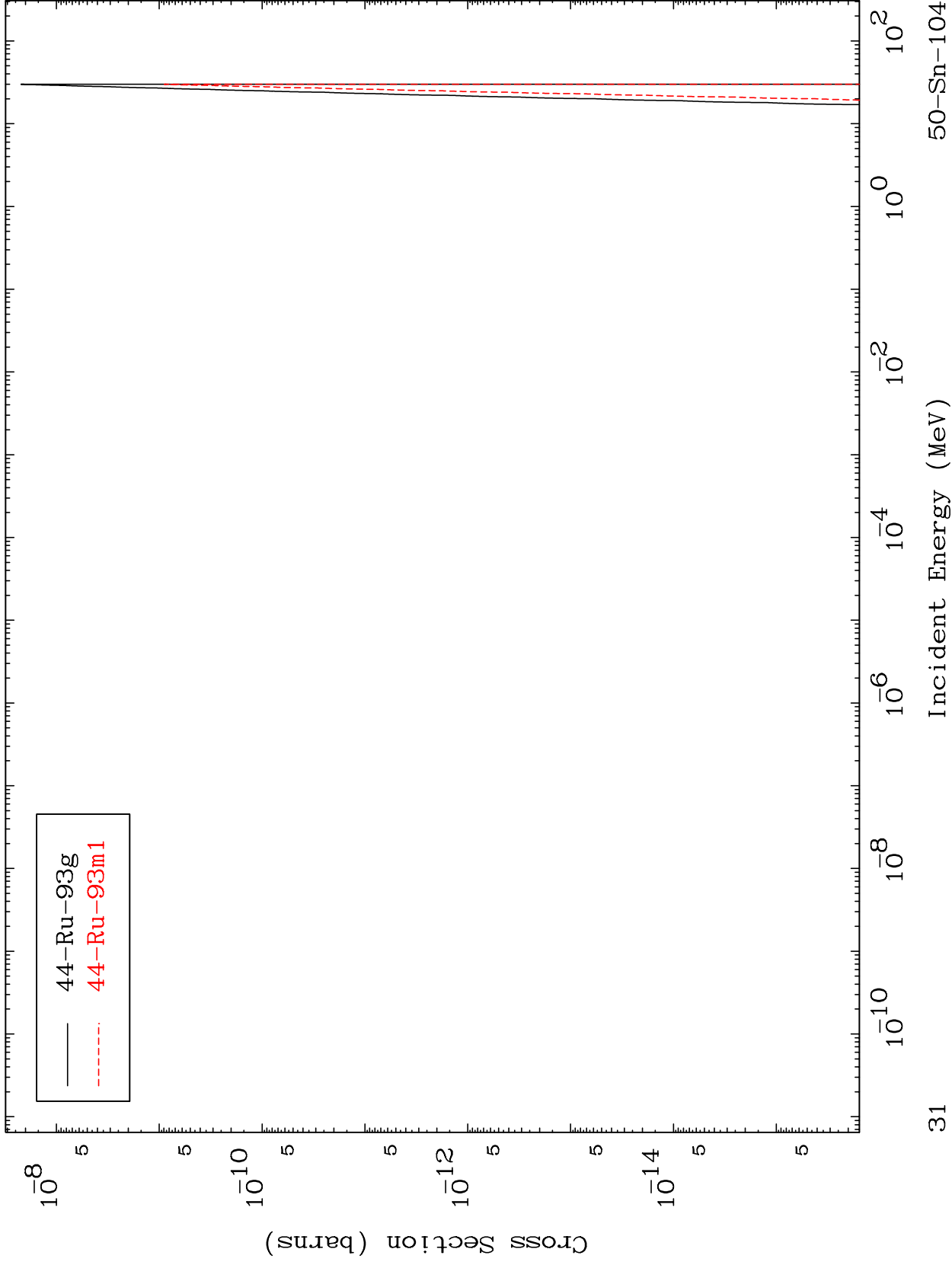
Incident Energy (MeV)

30

MAT 5001

50-Sn-104

(n,3 $\alpha$ )  
Radionuclide Production Cross Section



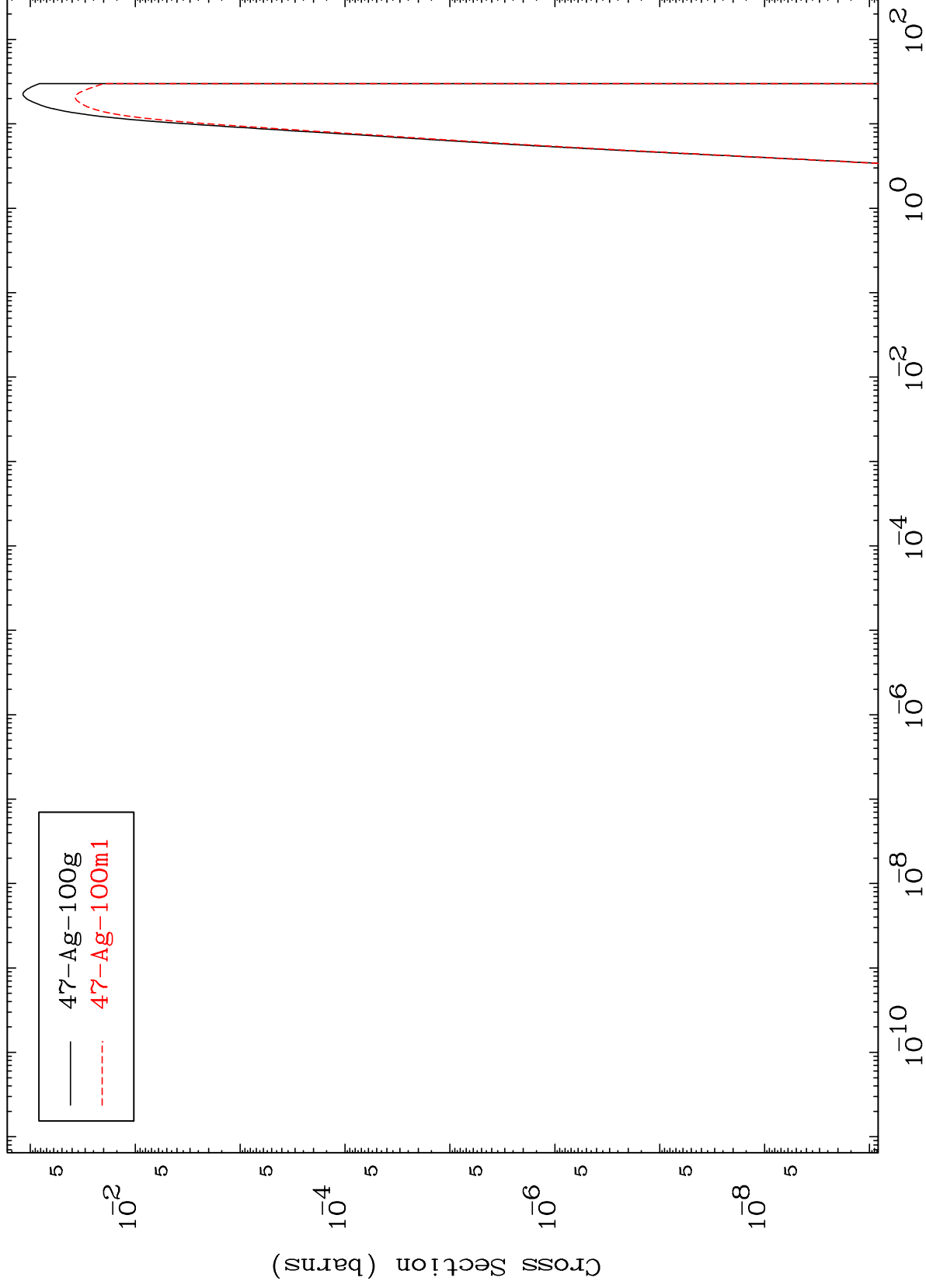


MAT 5001

(n,p)  $\alpha$

50-Sn-104

Radionuclide Production Cross Section



32

Incident Energy (MeV)

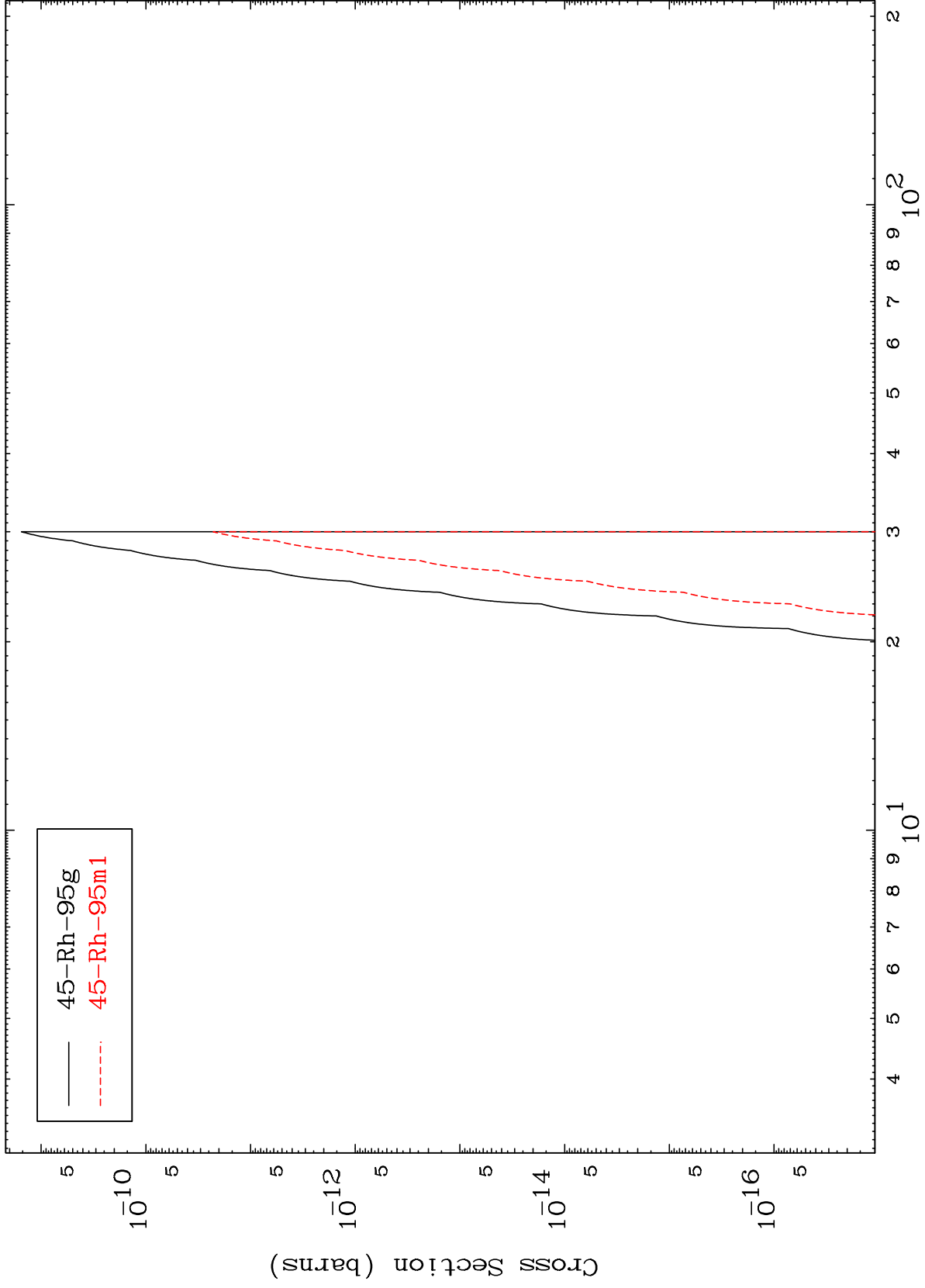
50-Sn-104

MAT 5001

(n,d) 2 $\alpha$

50-Sn-104

Radionuclide Production Cross Section



33

Incident Energy (MeV)

50-Sn-104

