

Program EVALPLOT
(Version 2021-1)

by

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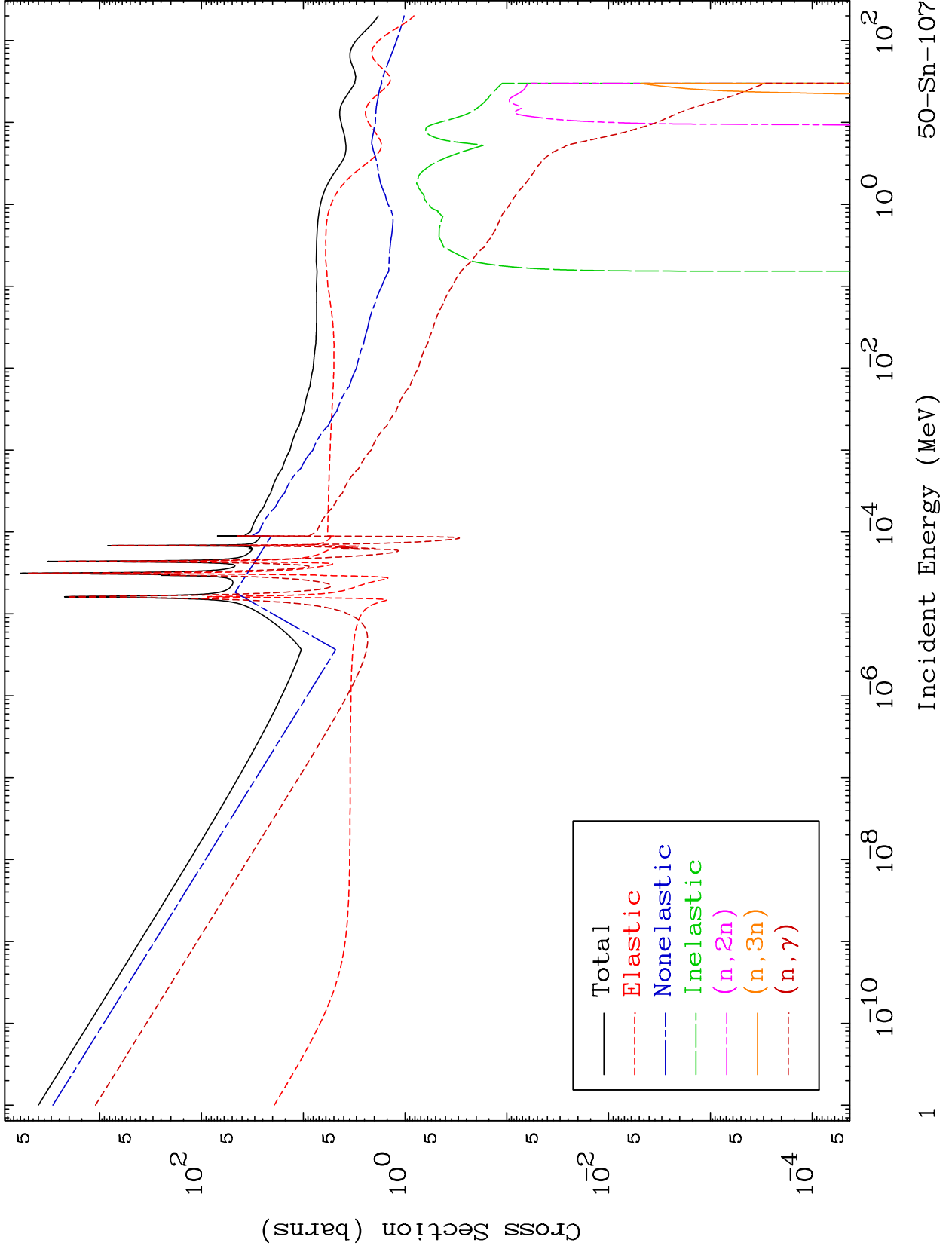
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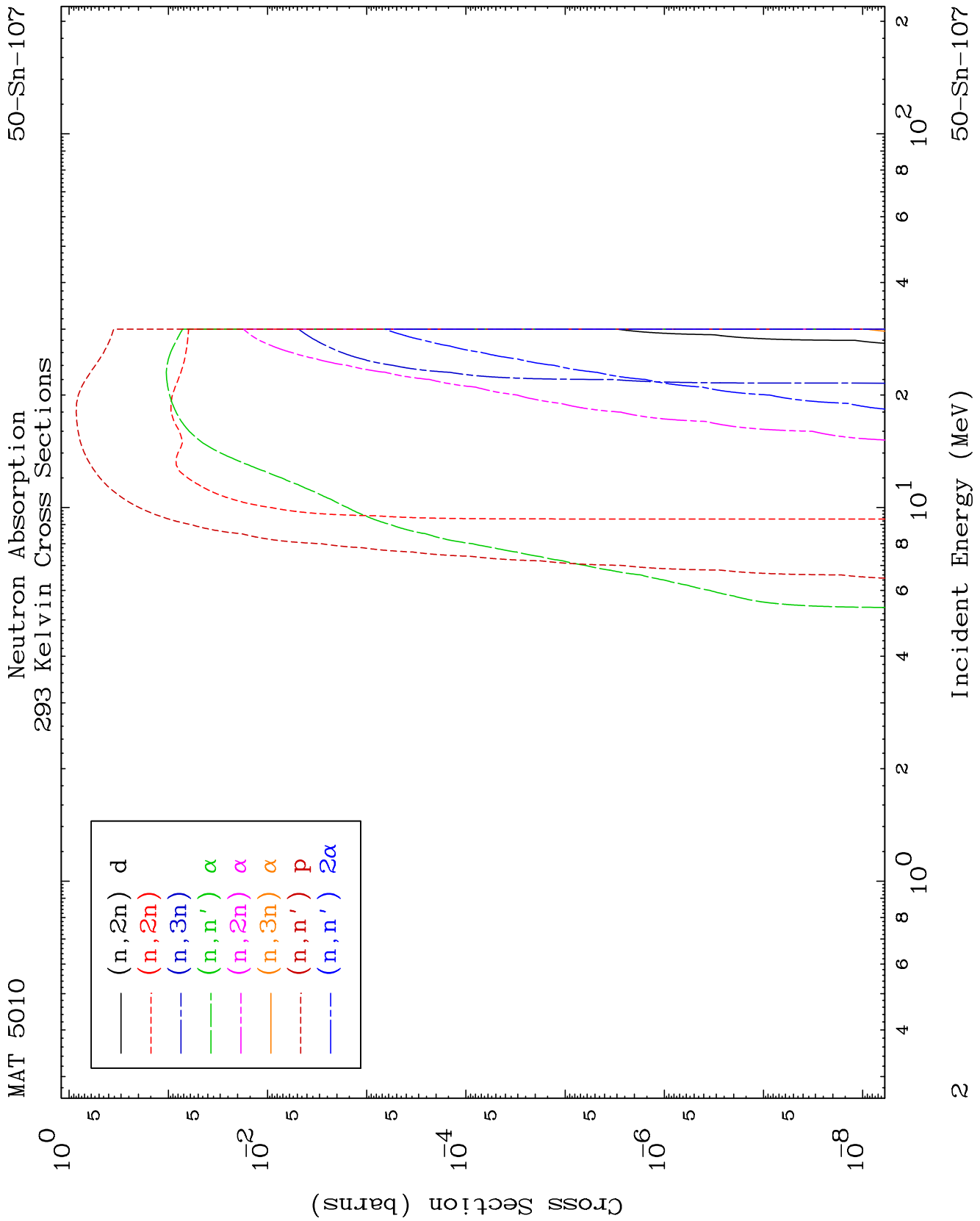
Press Mouse Button to Start

MAT 5010

Neutron Major
293 Kelvin Cross Sections

50-Sn-107

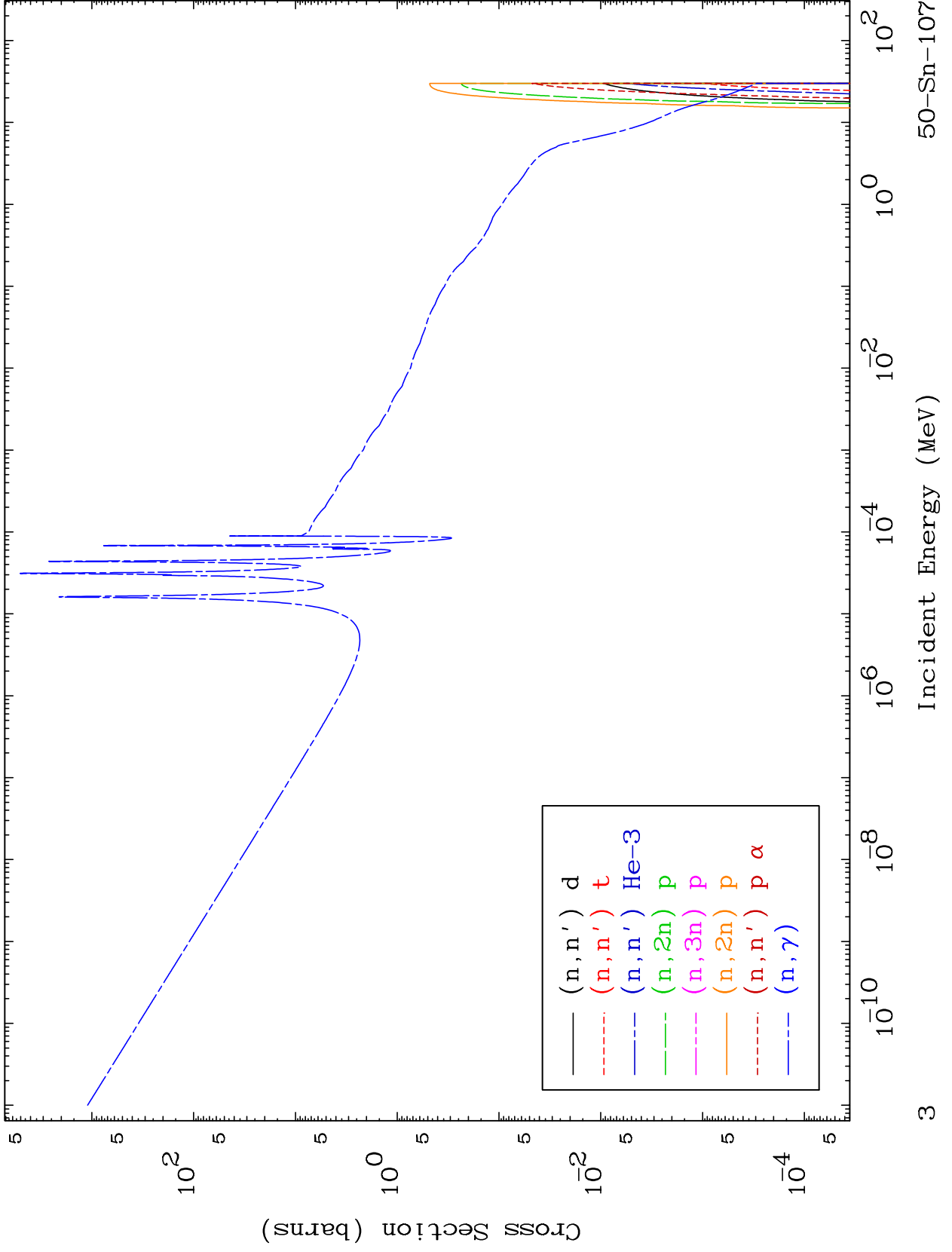


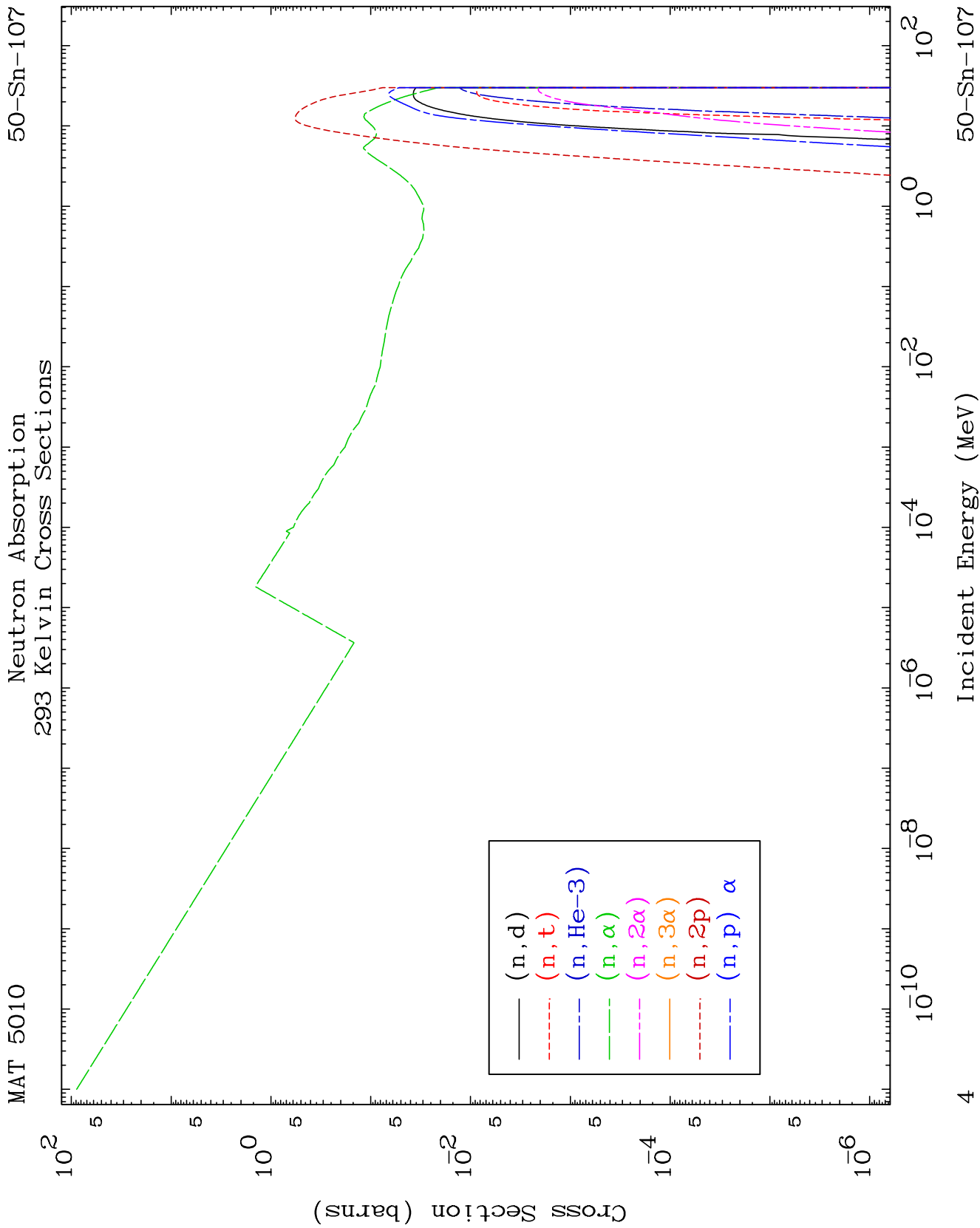


MAT 5010

Neutron Absorption
293 Kelvin Cross Sections

50-Sn-107

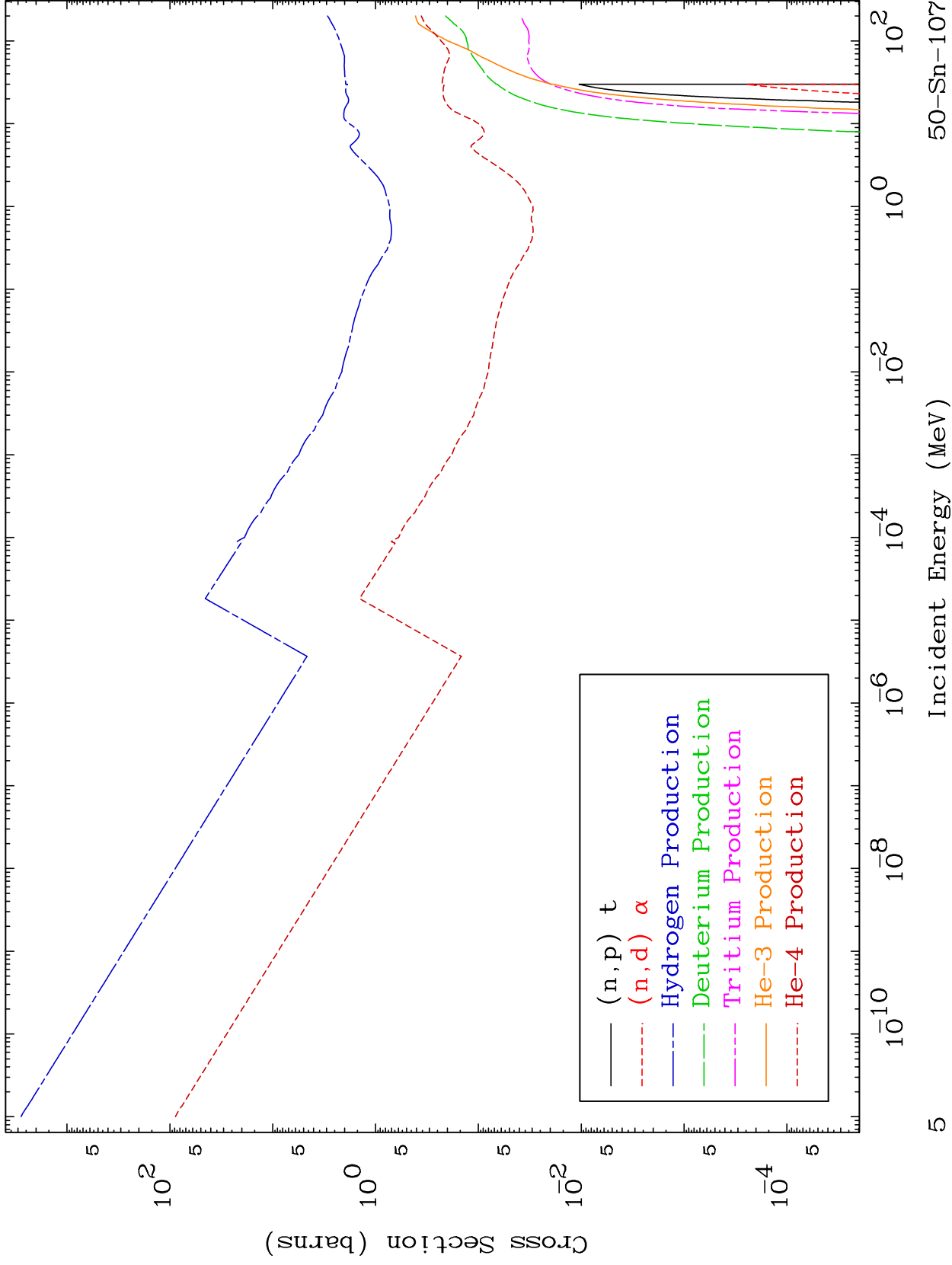


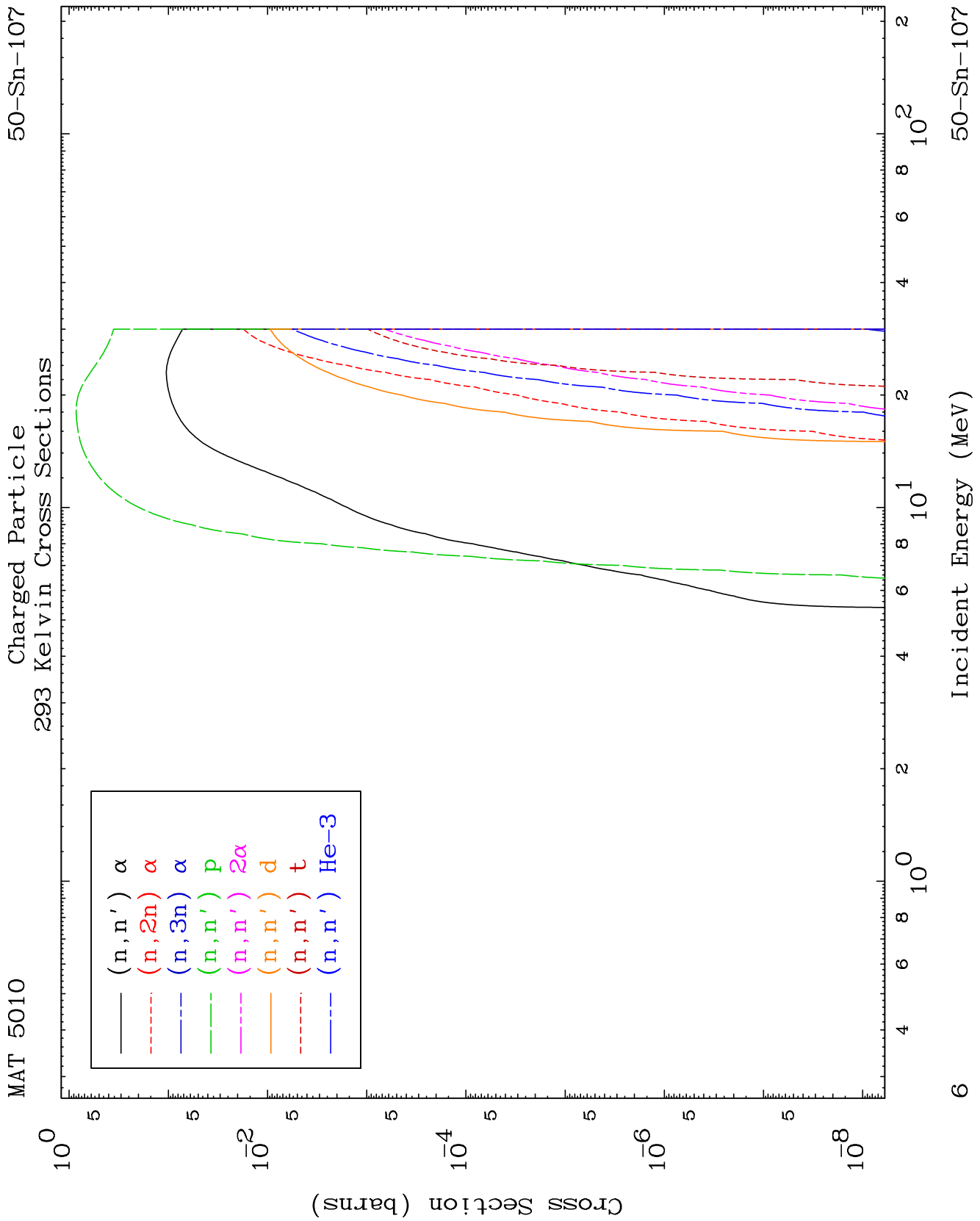


MAT 5010

Neutron Absorption
293 Kelvin Cross Sections

50-Sn-107

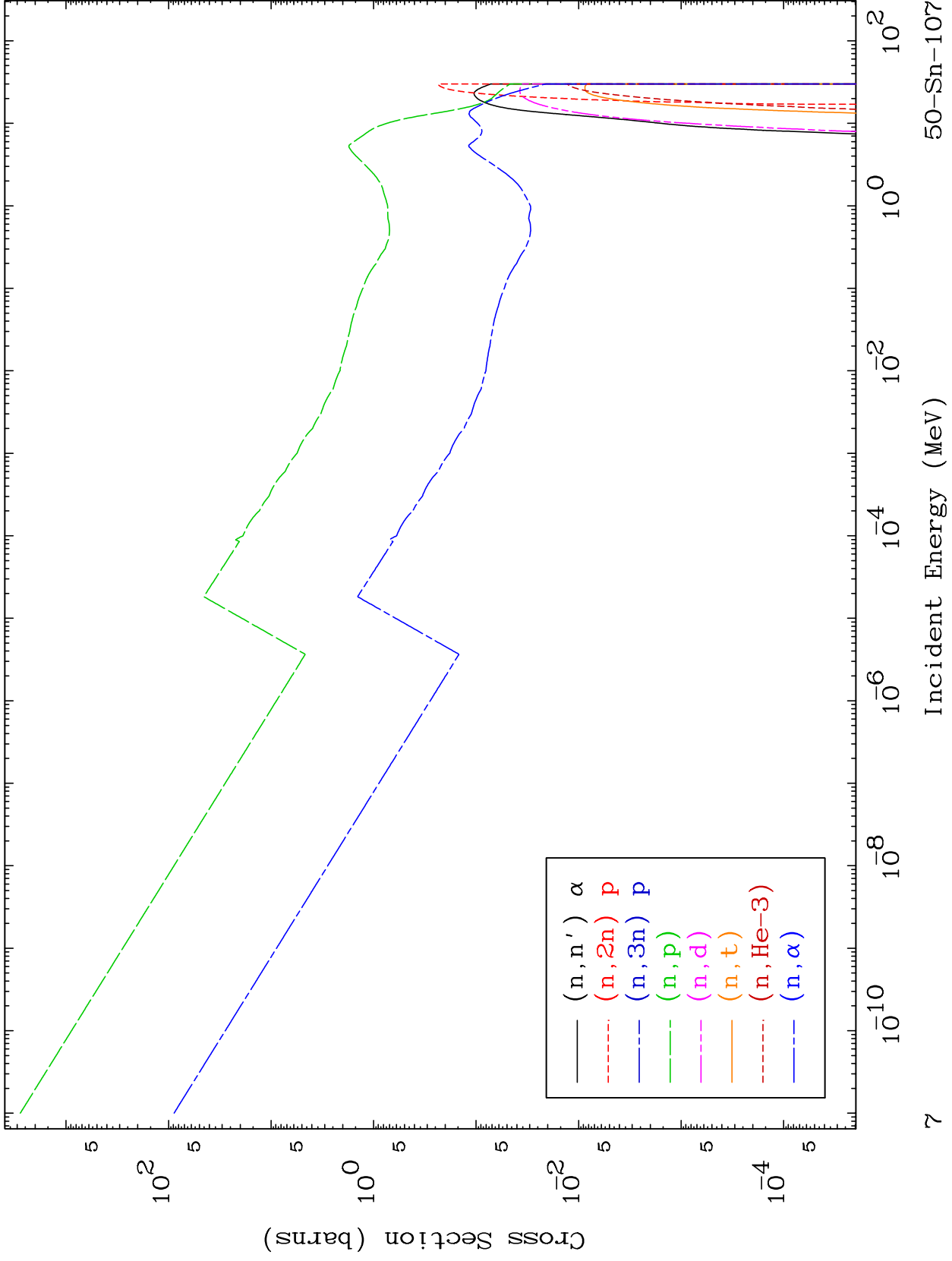




MAT 5010

Charged Particle
293 Kelvin Cross Sections

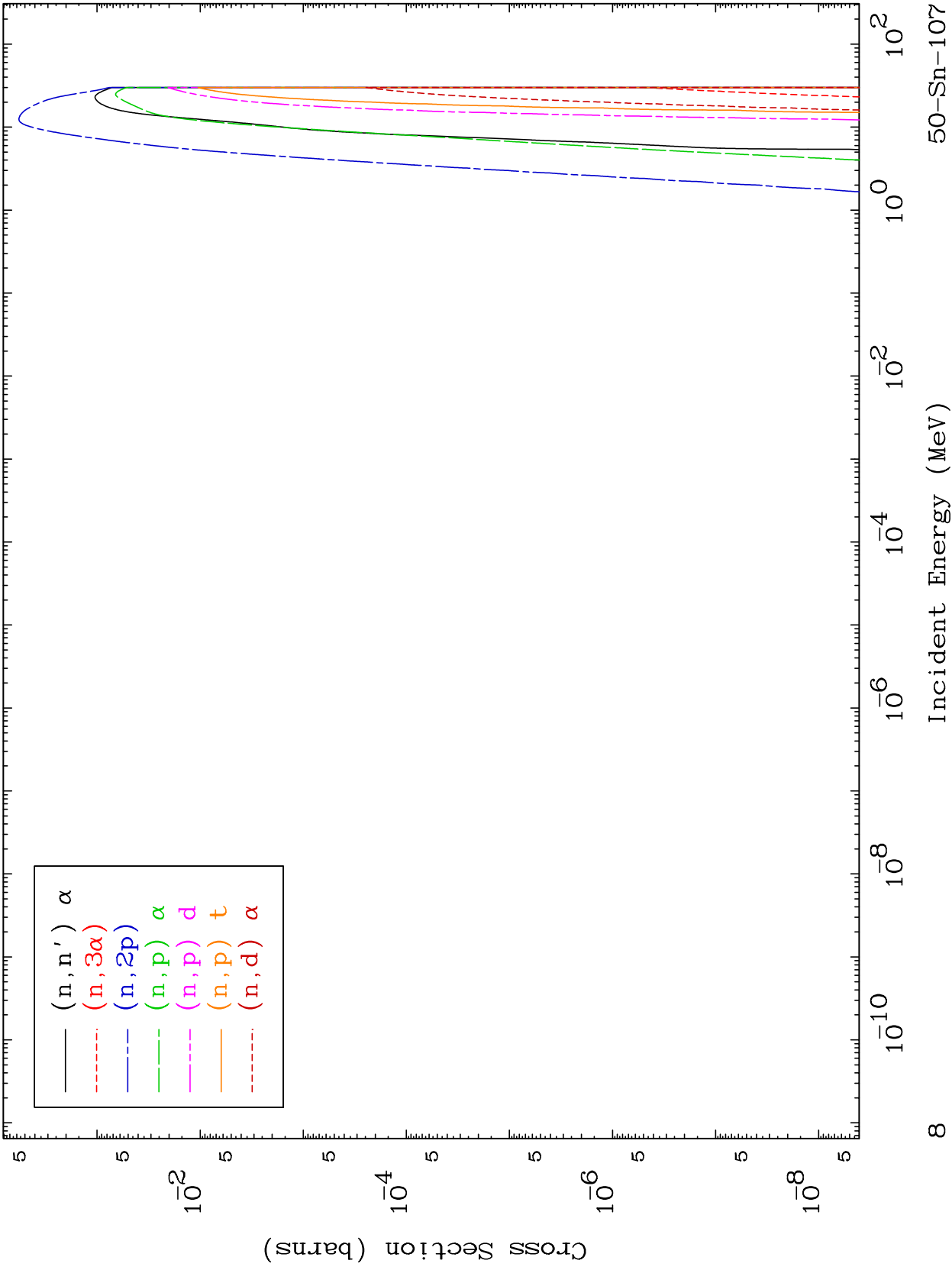
50-Sn-107



MAT 5010

Charged Particle
293 Kelvin Cross Sections

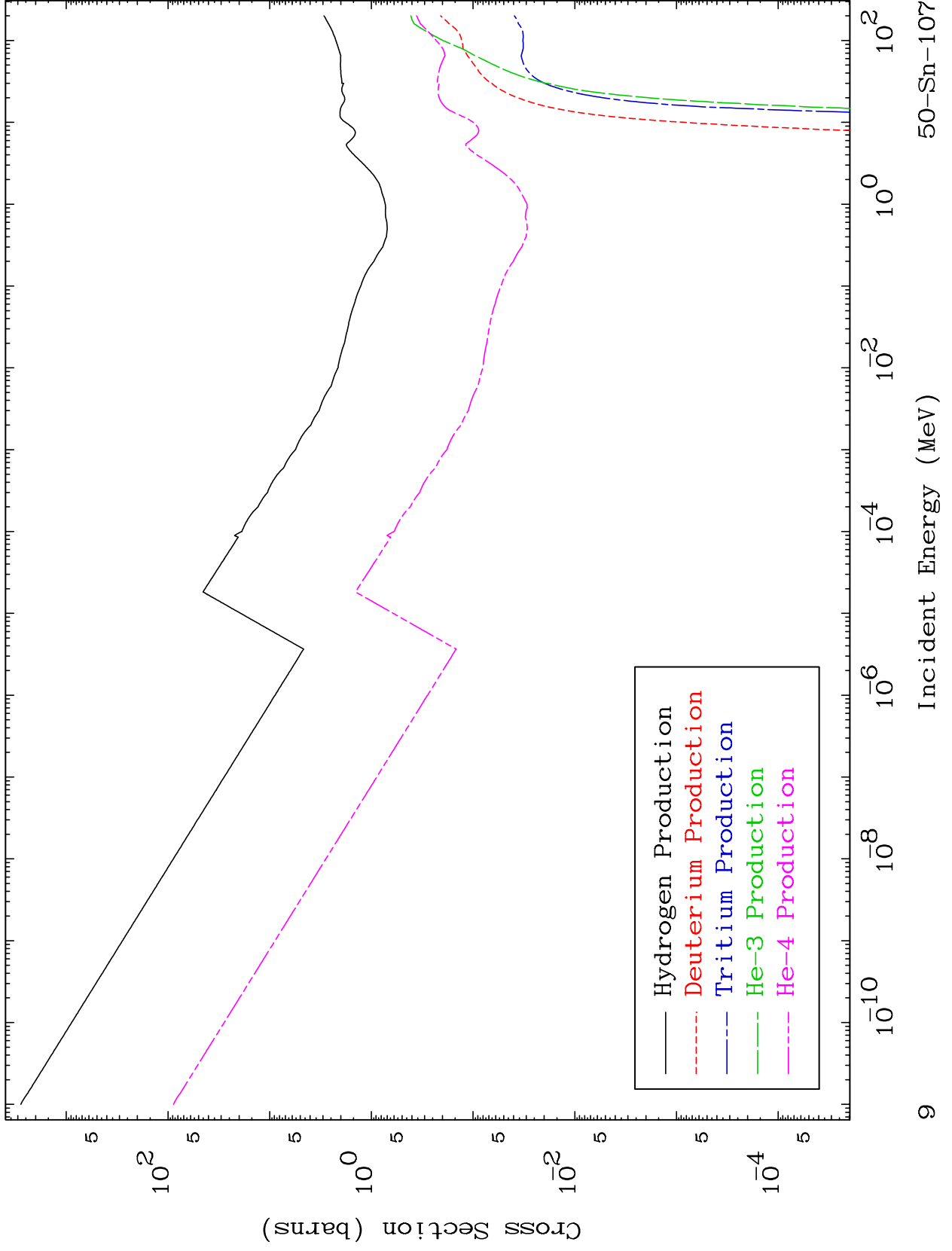
50-Sn-107



MAT 5010

Particle Production
293 Kelvin Cross Sections

50-Sn-107

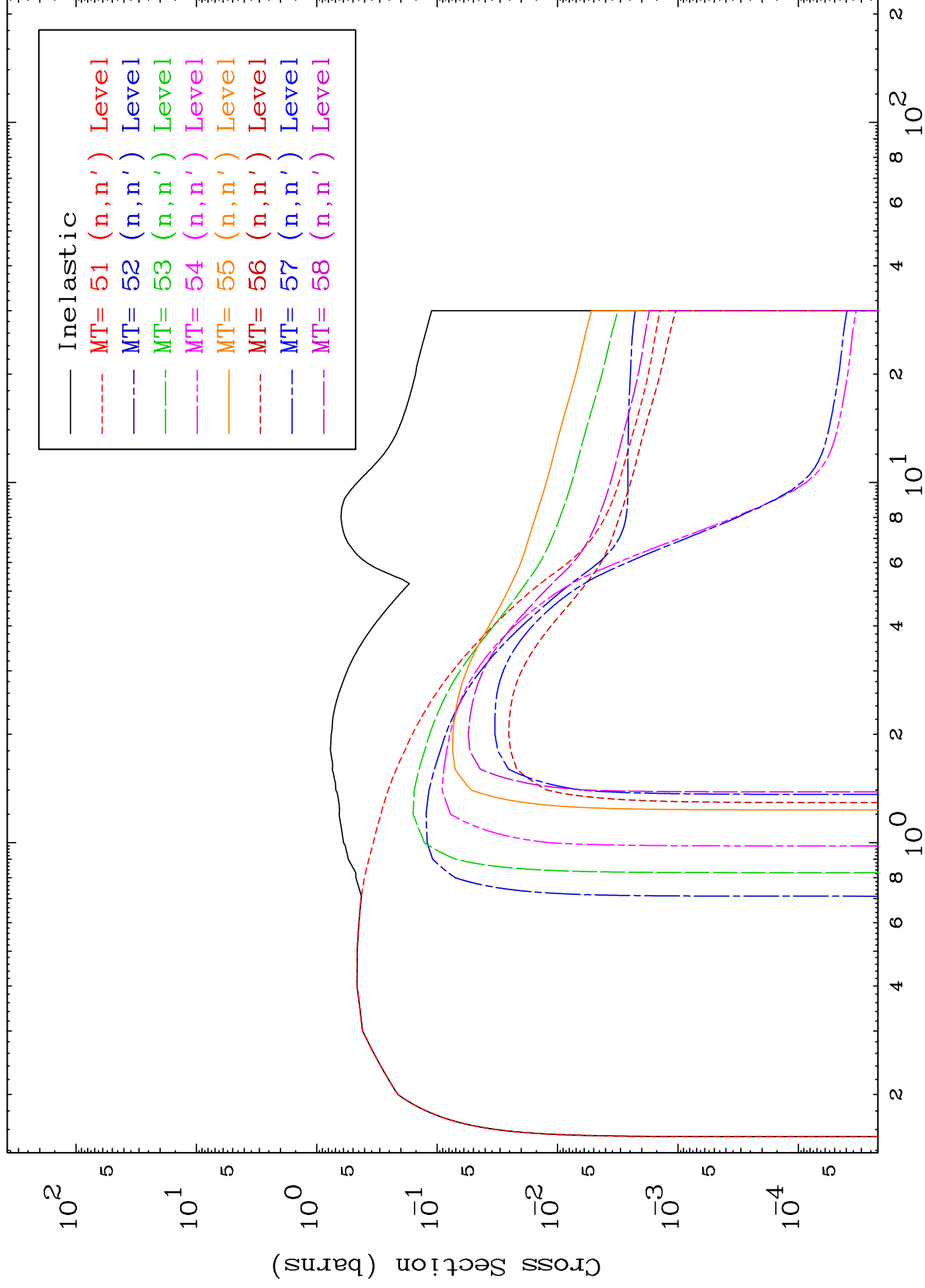


MAT 5010

(n,n') Levels

50-Sn-107

293 Kelvin Cross Sections



10

Incident Energy (MeV)

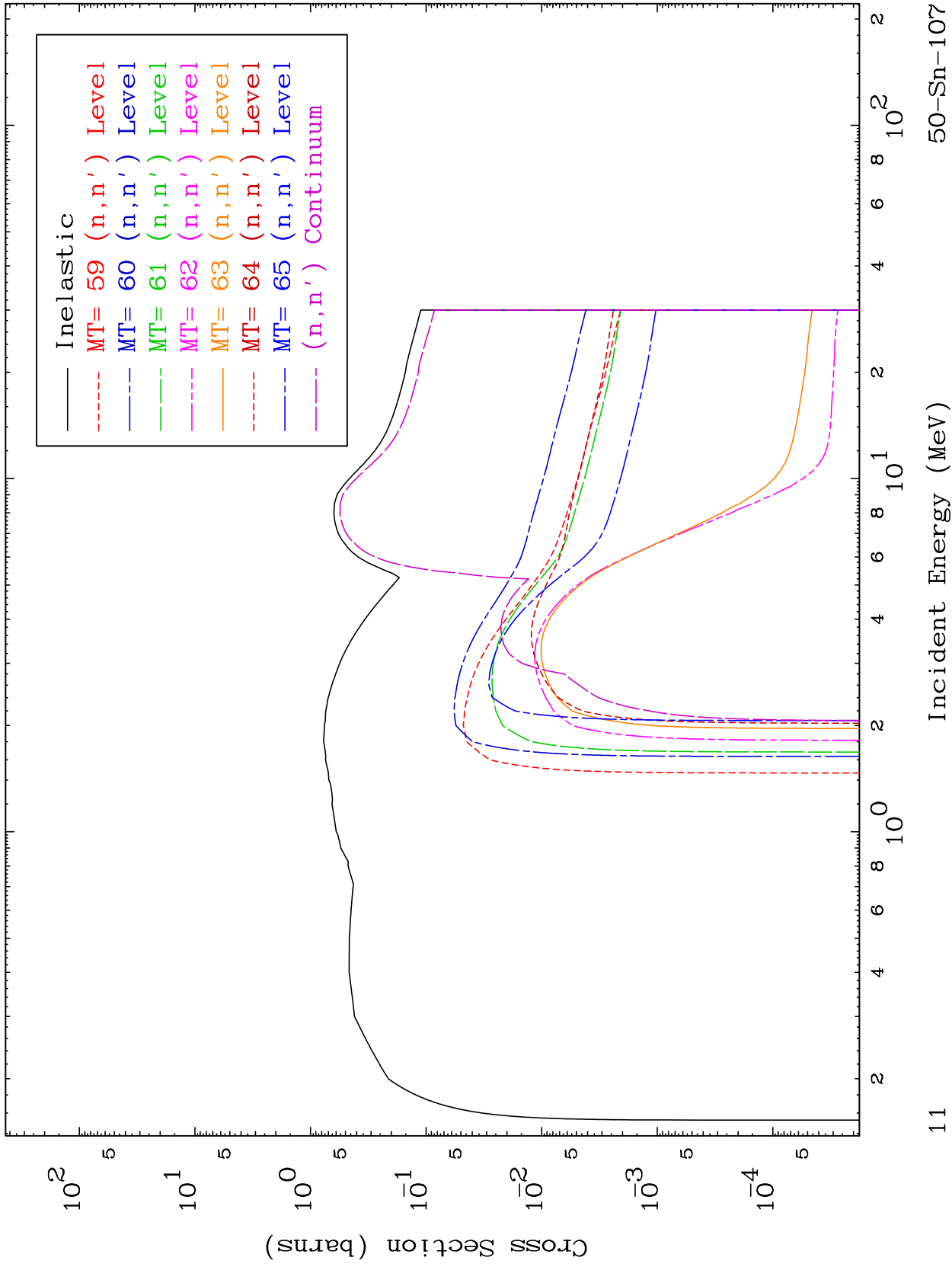
50-Sn-107

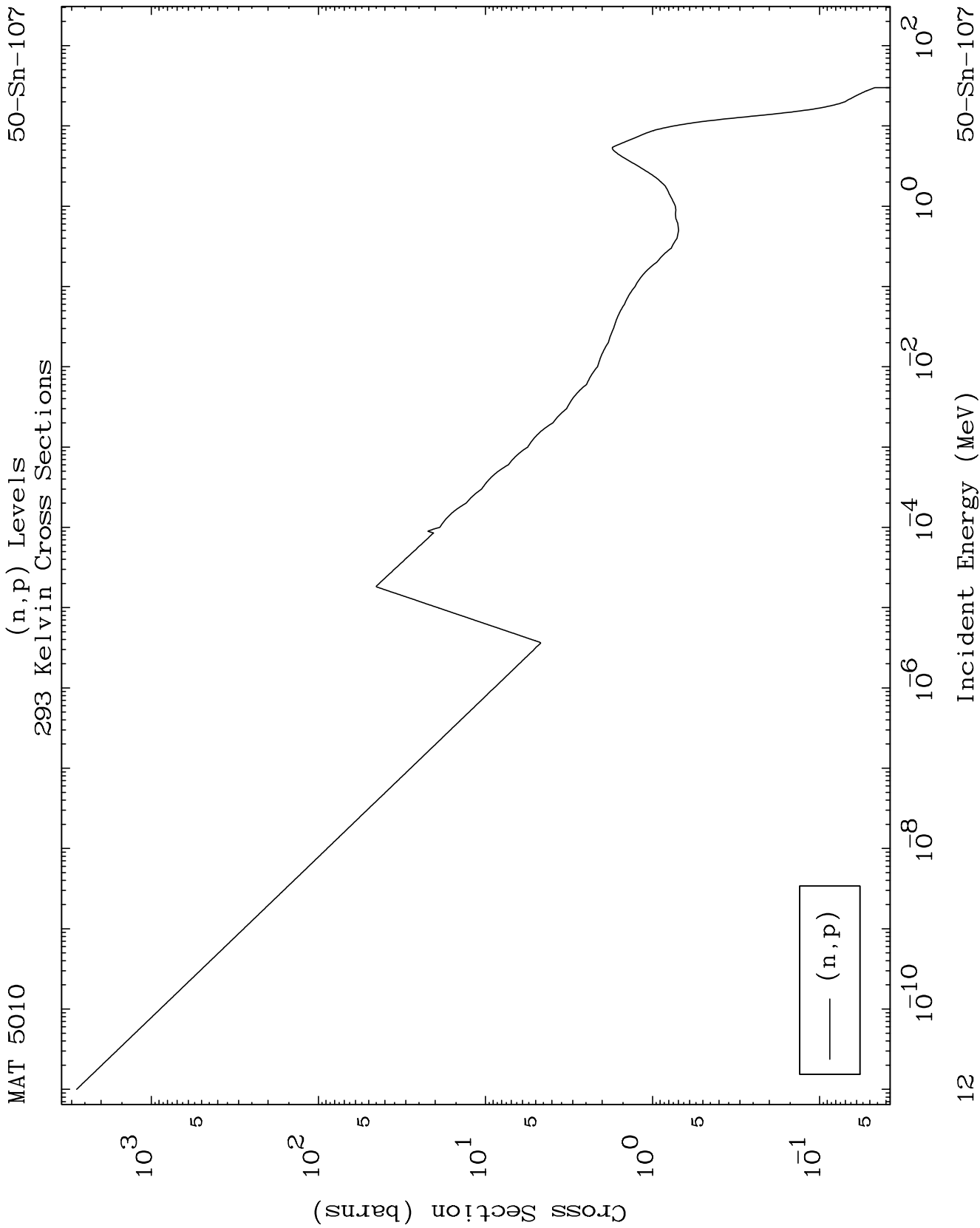
MAT 5010

(n,n') Levels

293 Kelvin Cross Sections

50-Sn-107

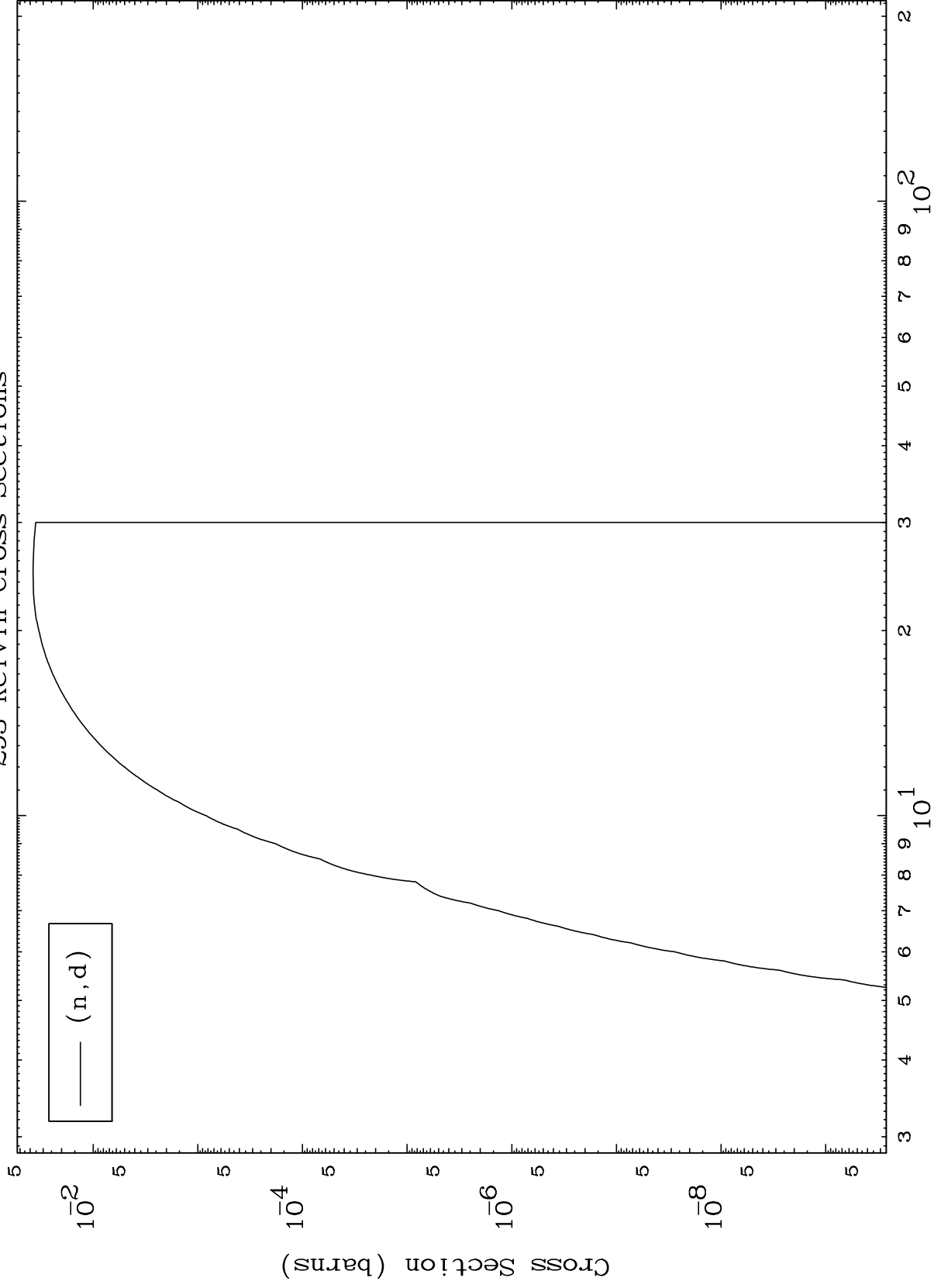




MAT 5010

(n,d) Levels
293 Kelvin Cross Sections

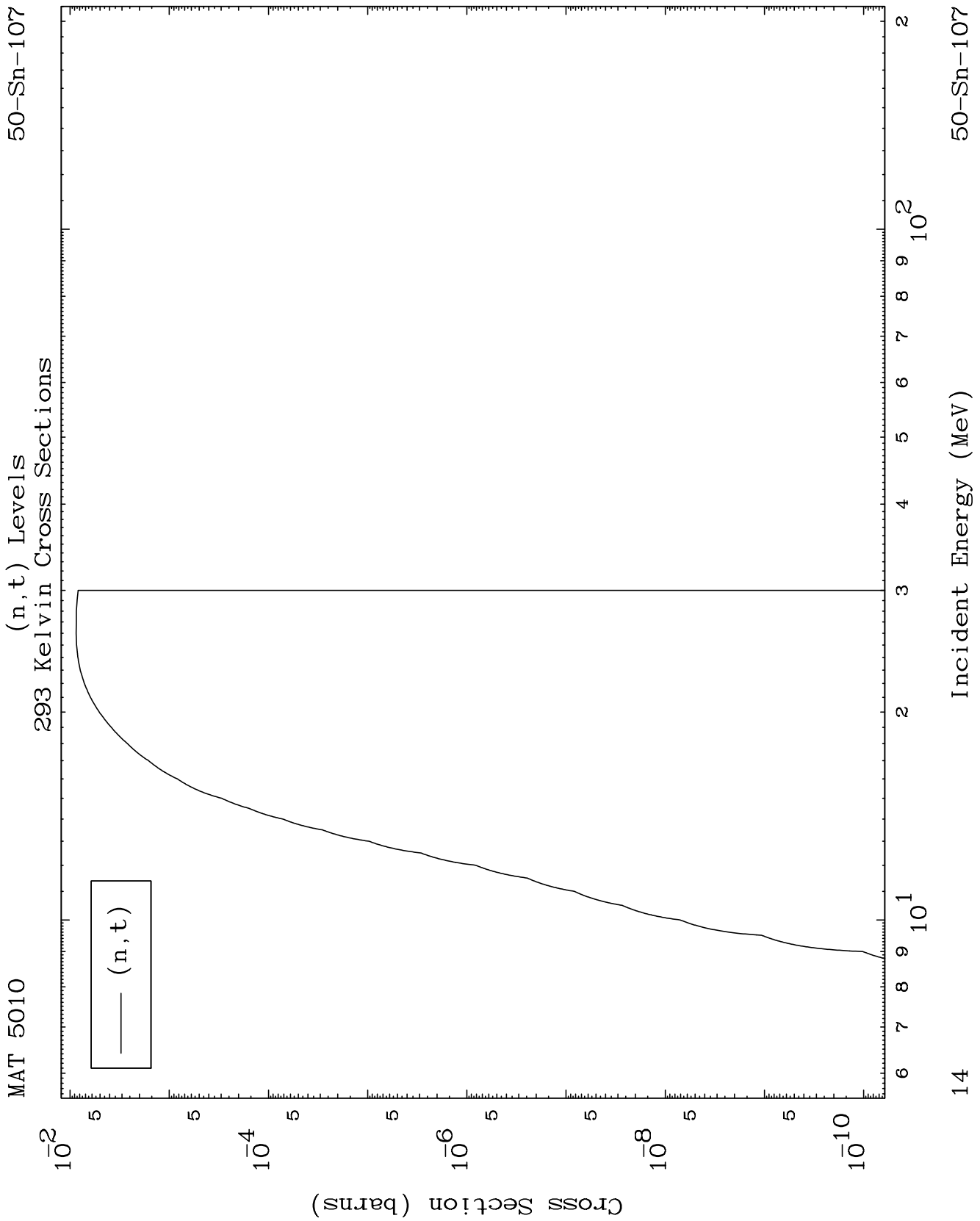
50-Sn-107



13

Incident Energy (MeV)

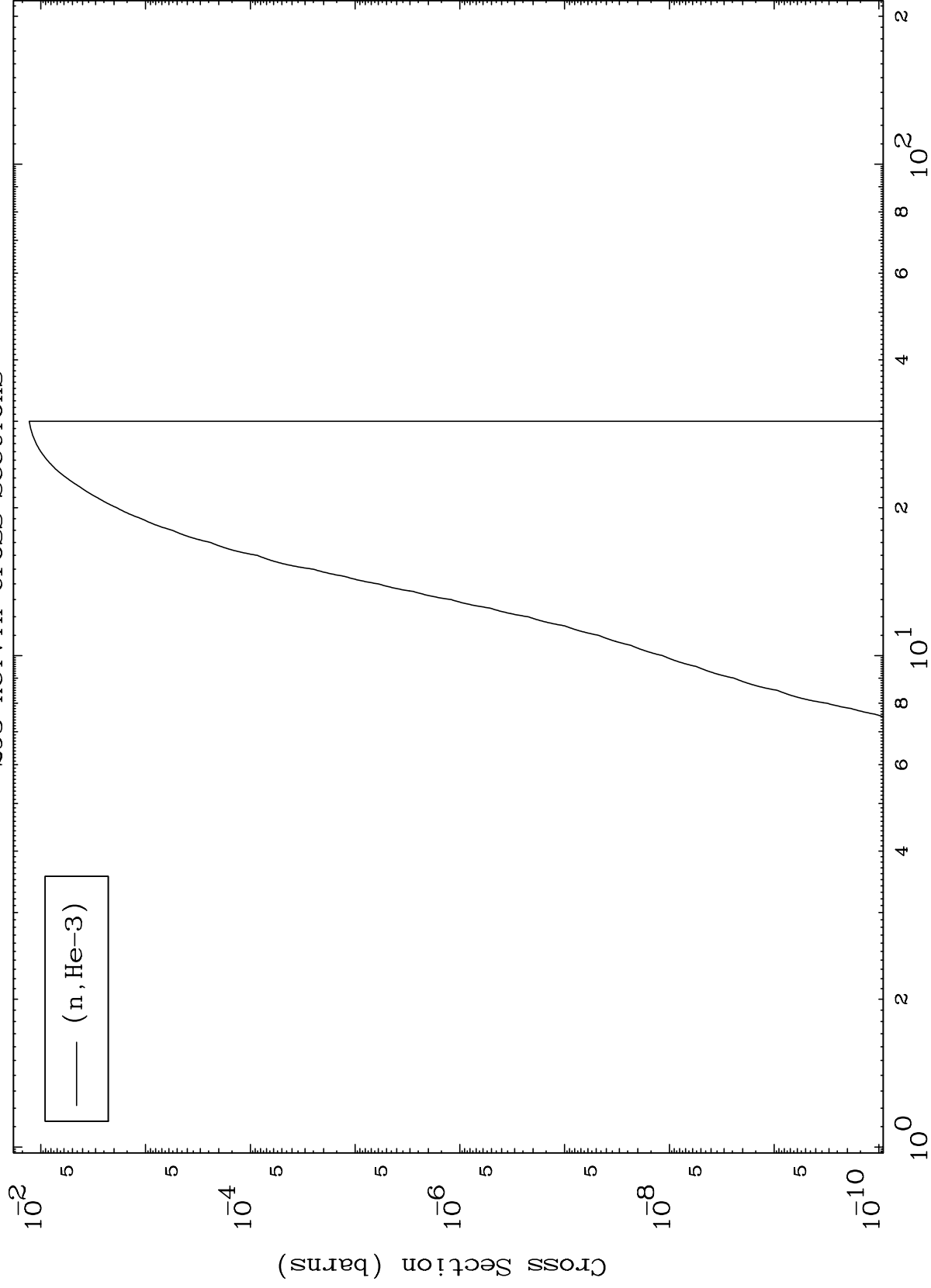
50-Sn-107



MAT 5010

(n,He3) Levels
293 Kelvin Cross Sections

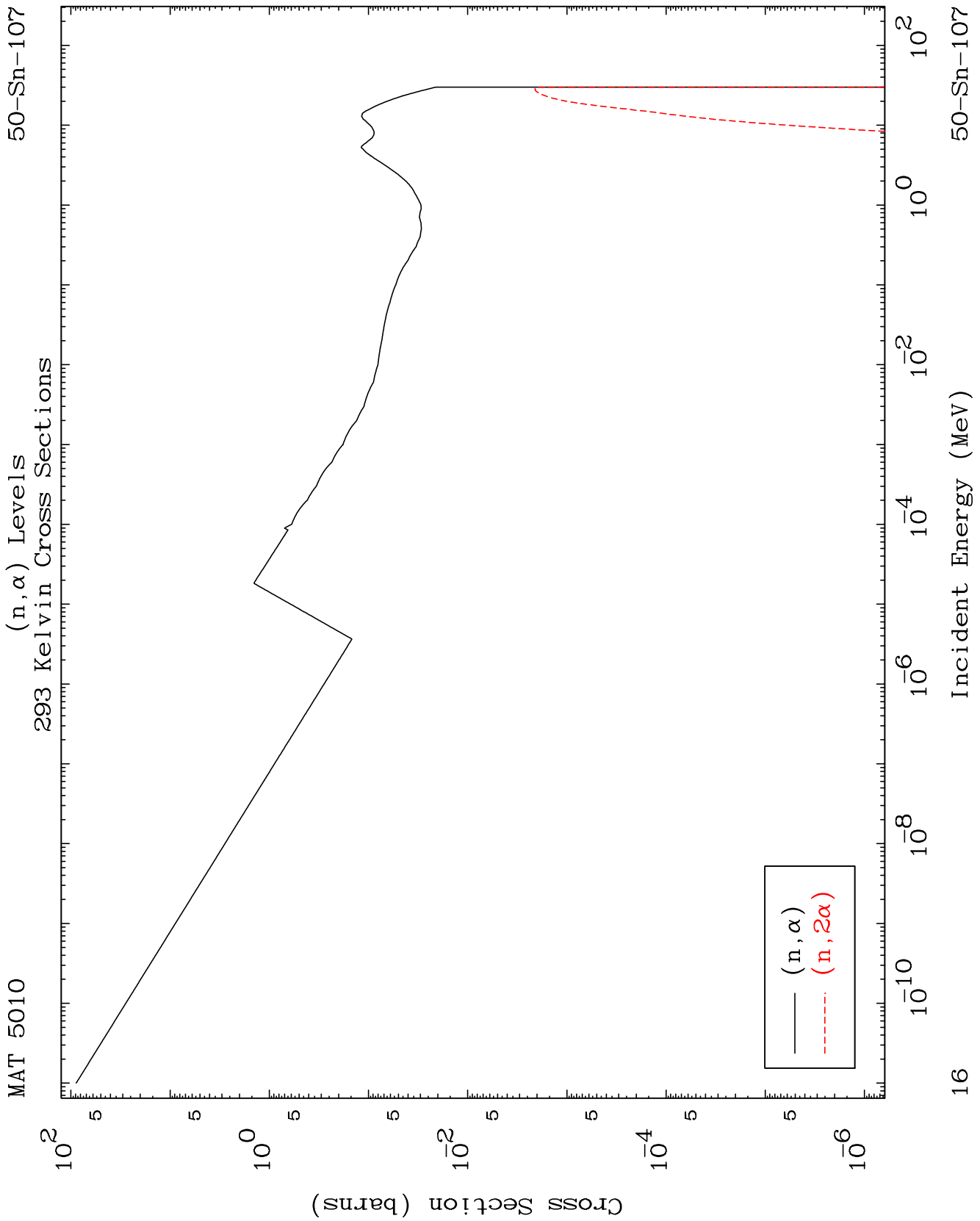
50-Sn-107

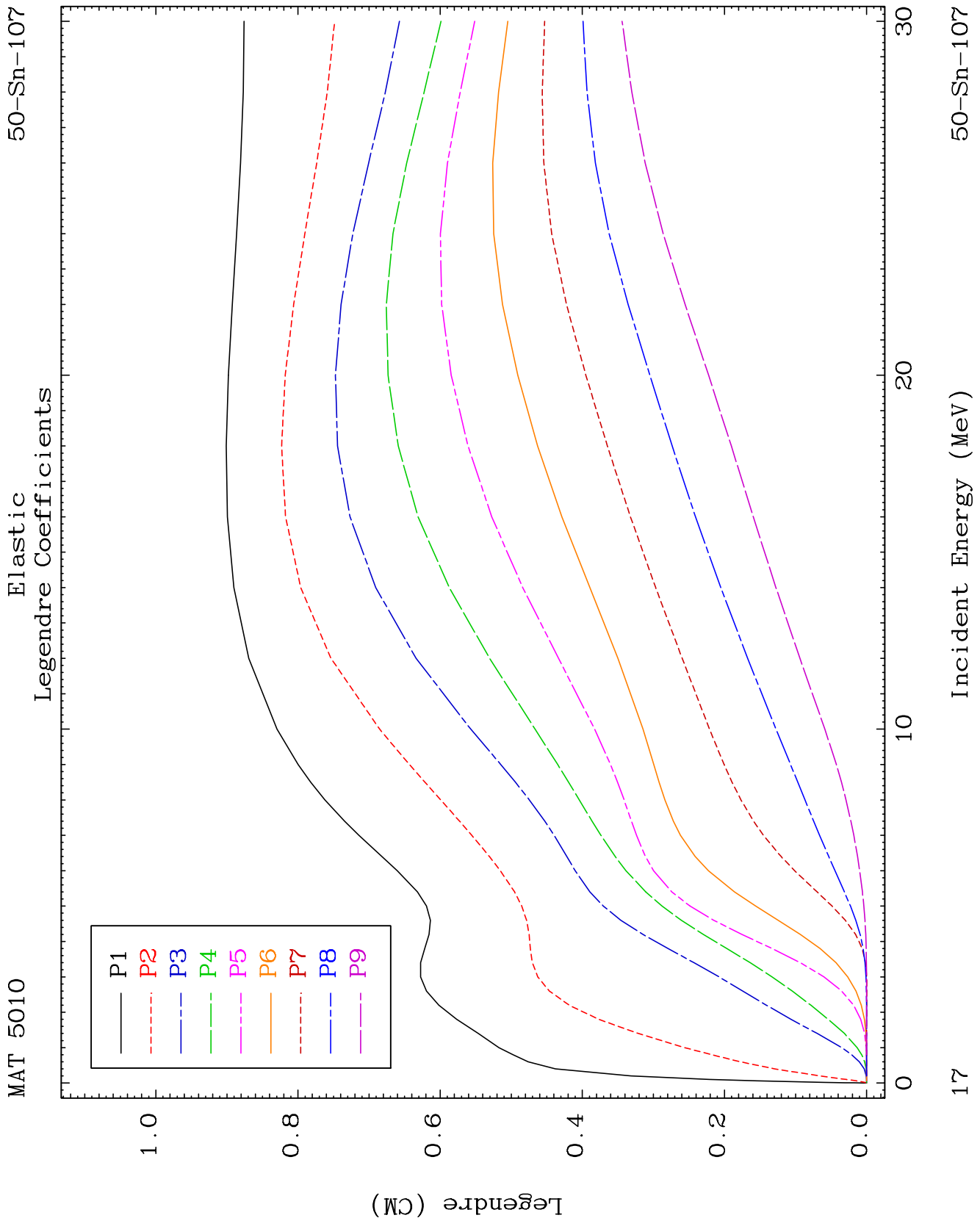


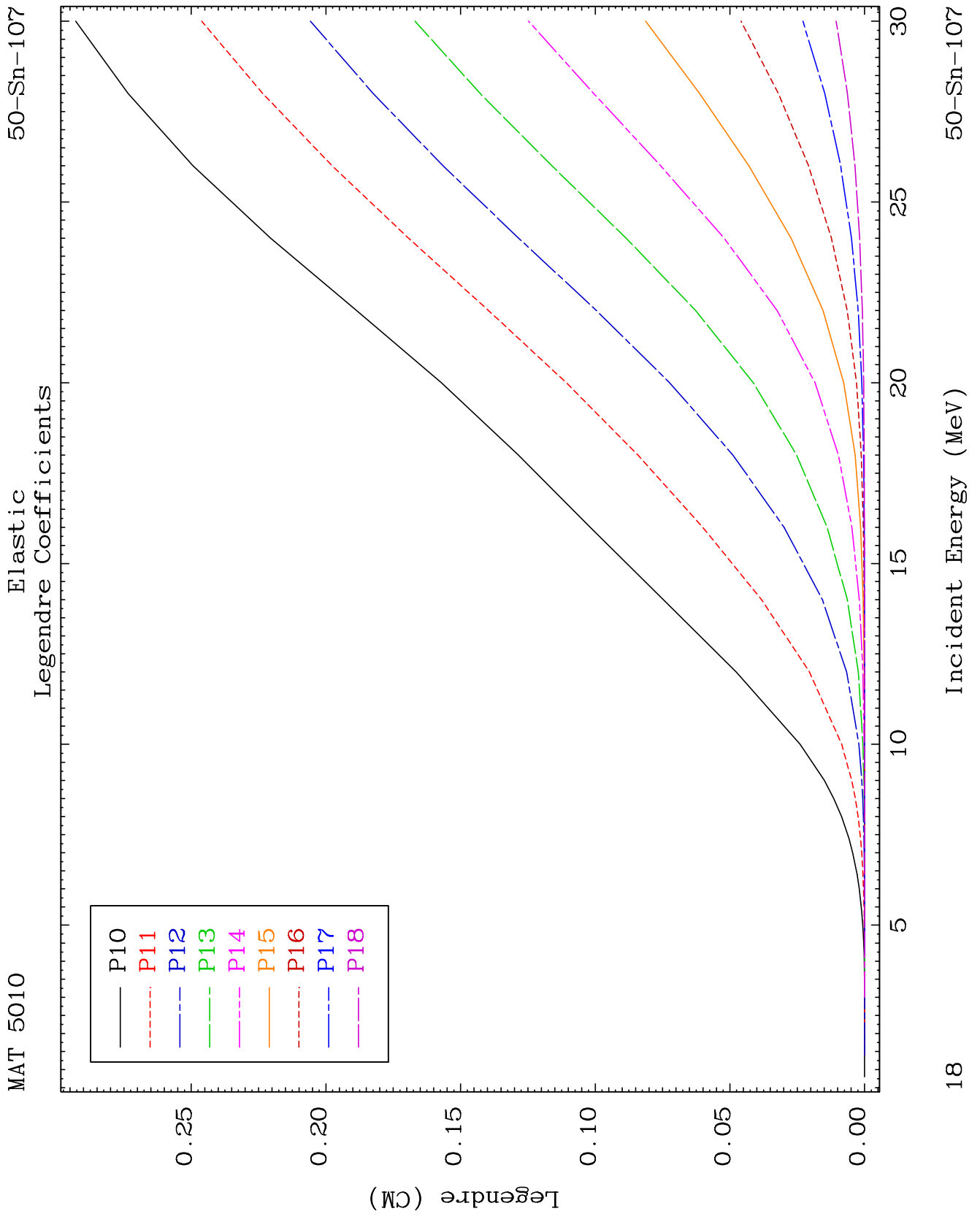
Incident Energy (MeV)

50-Sn-107

15



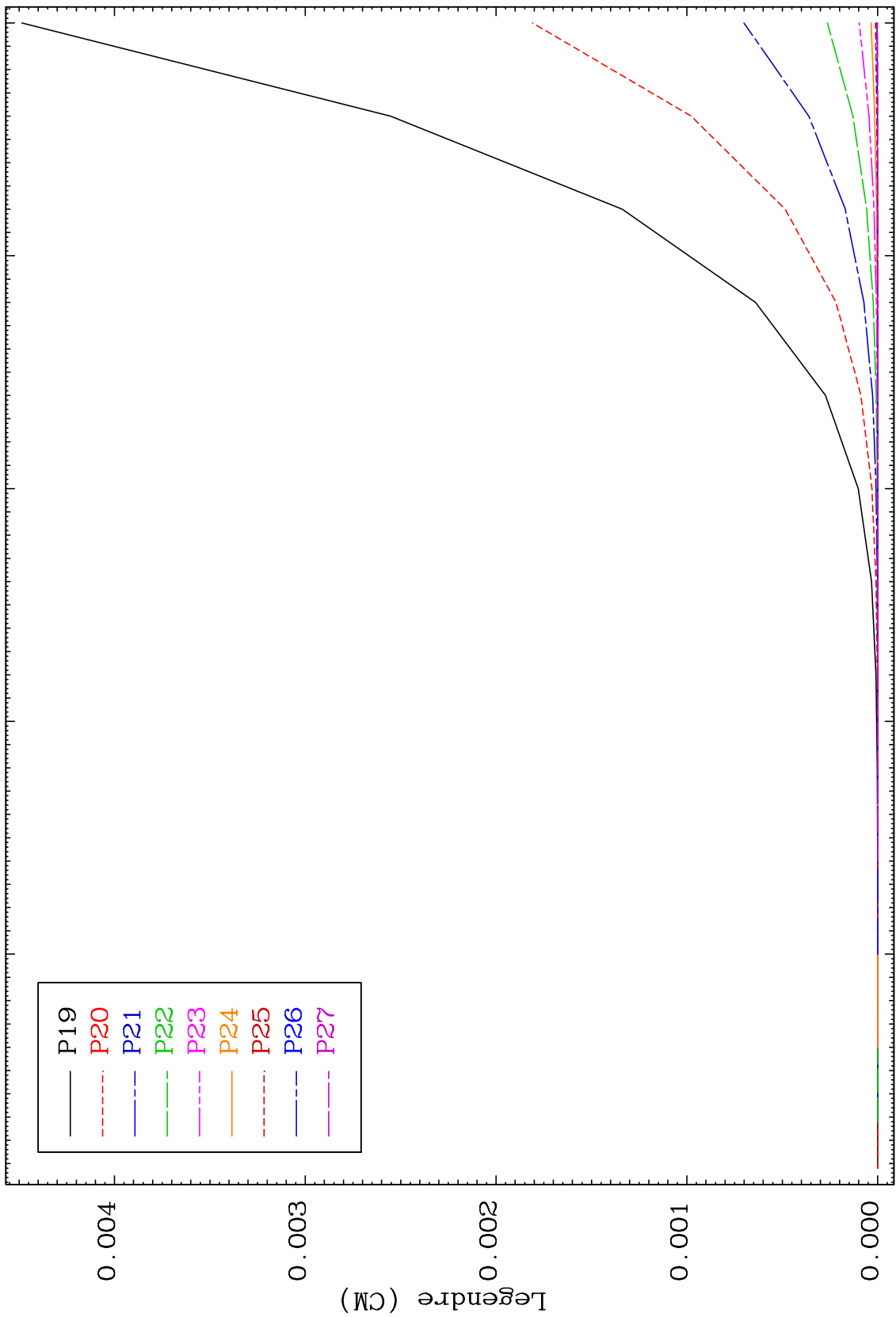




MAT 5010

Elastic Legendre Coefficients

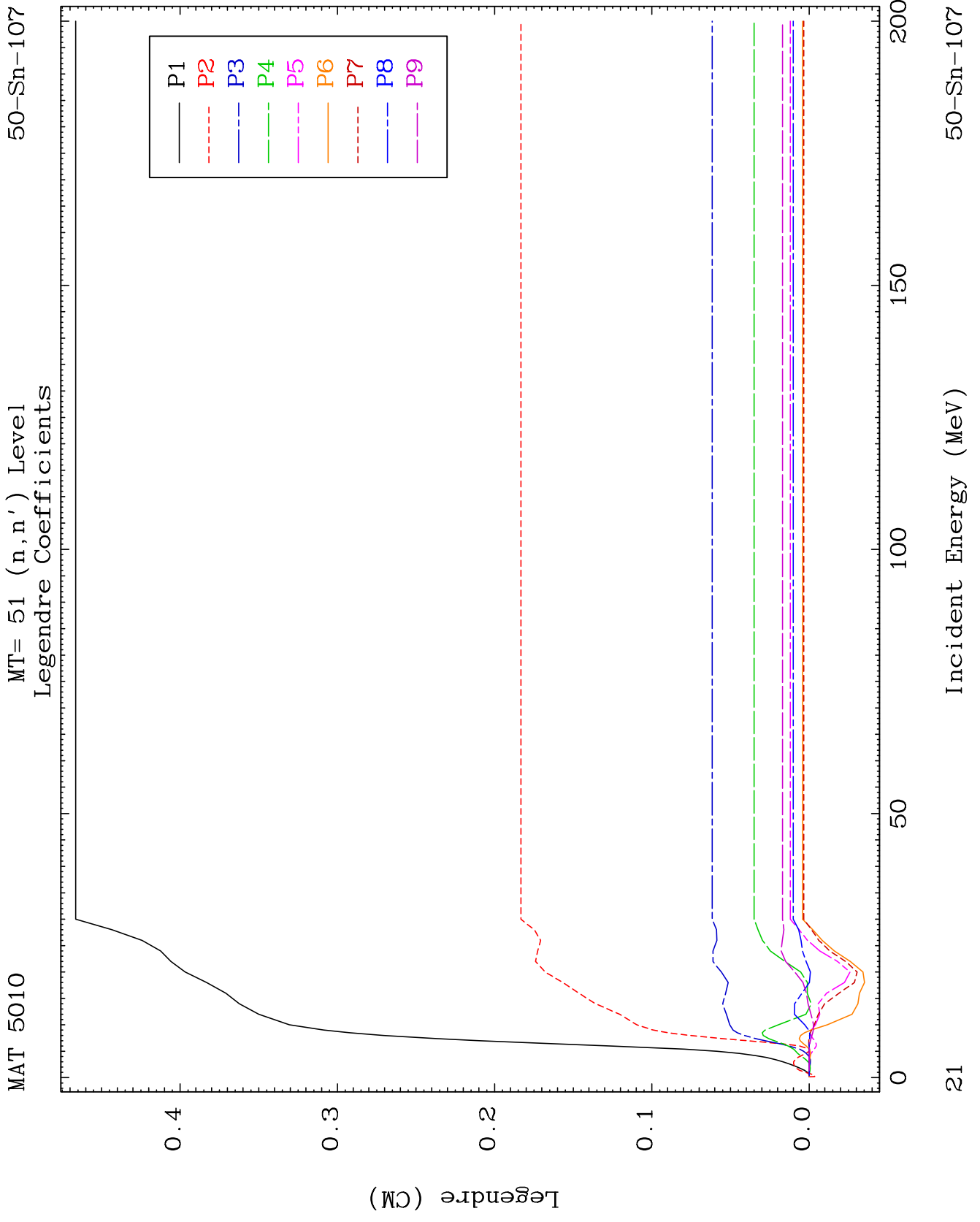
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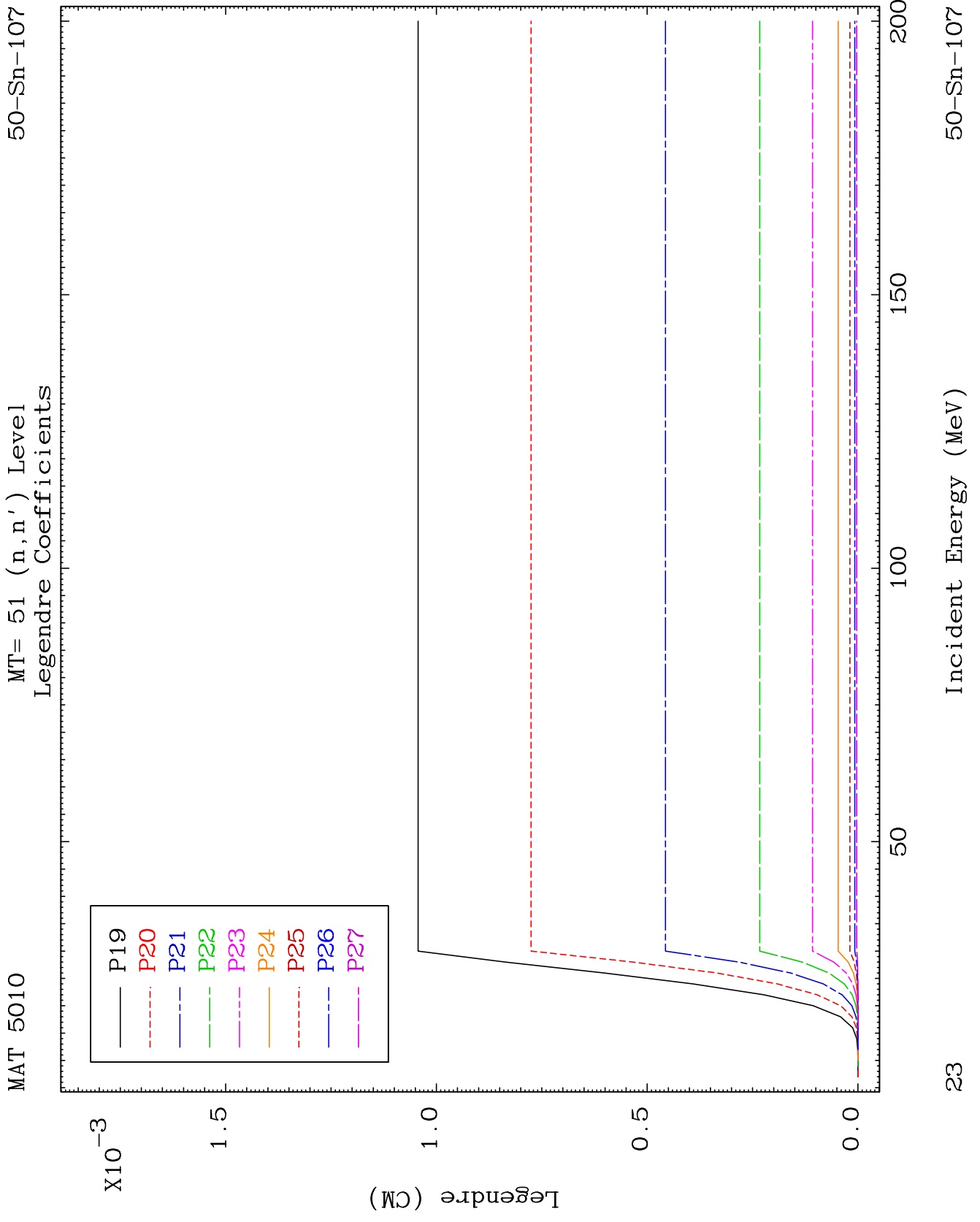


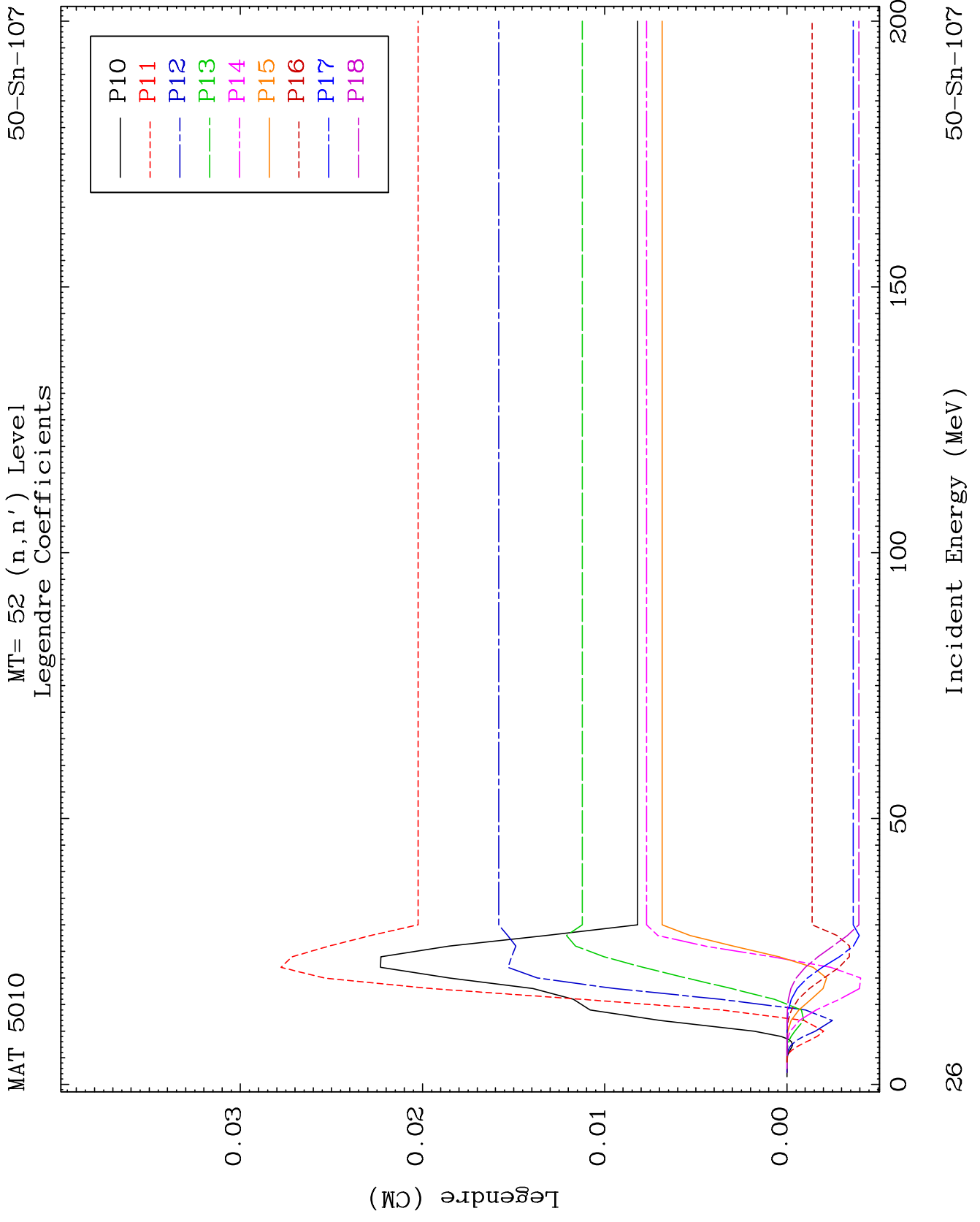
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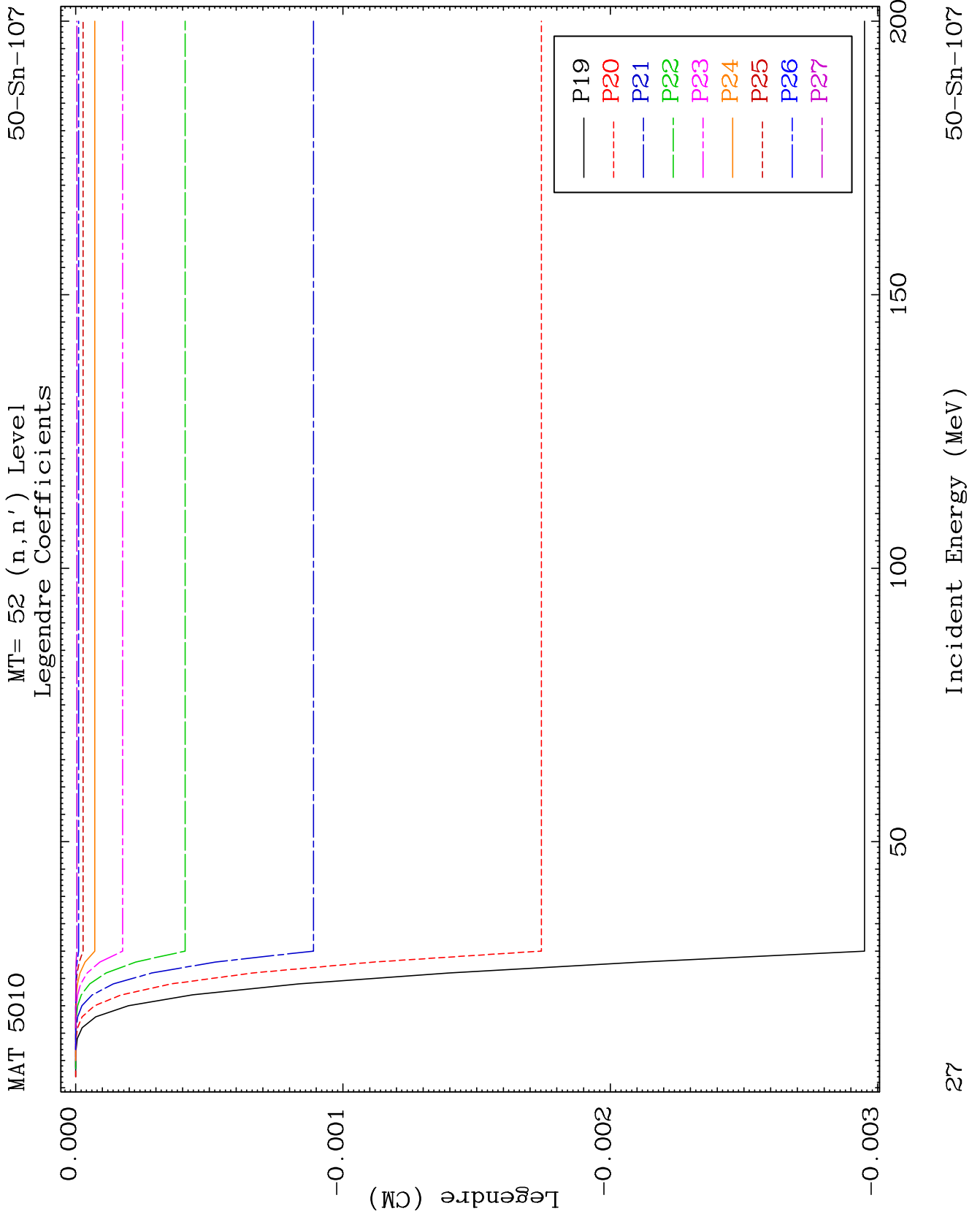
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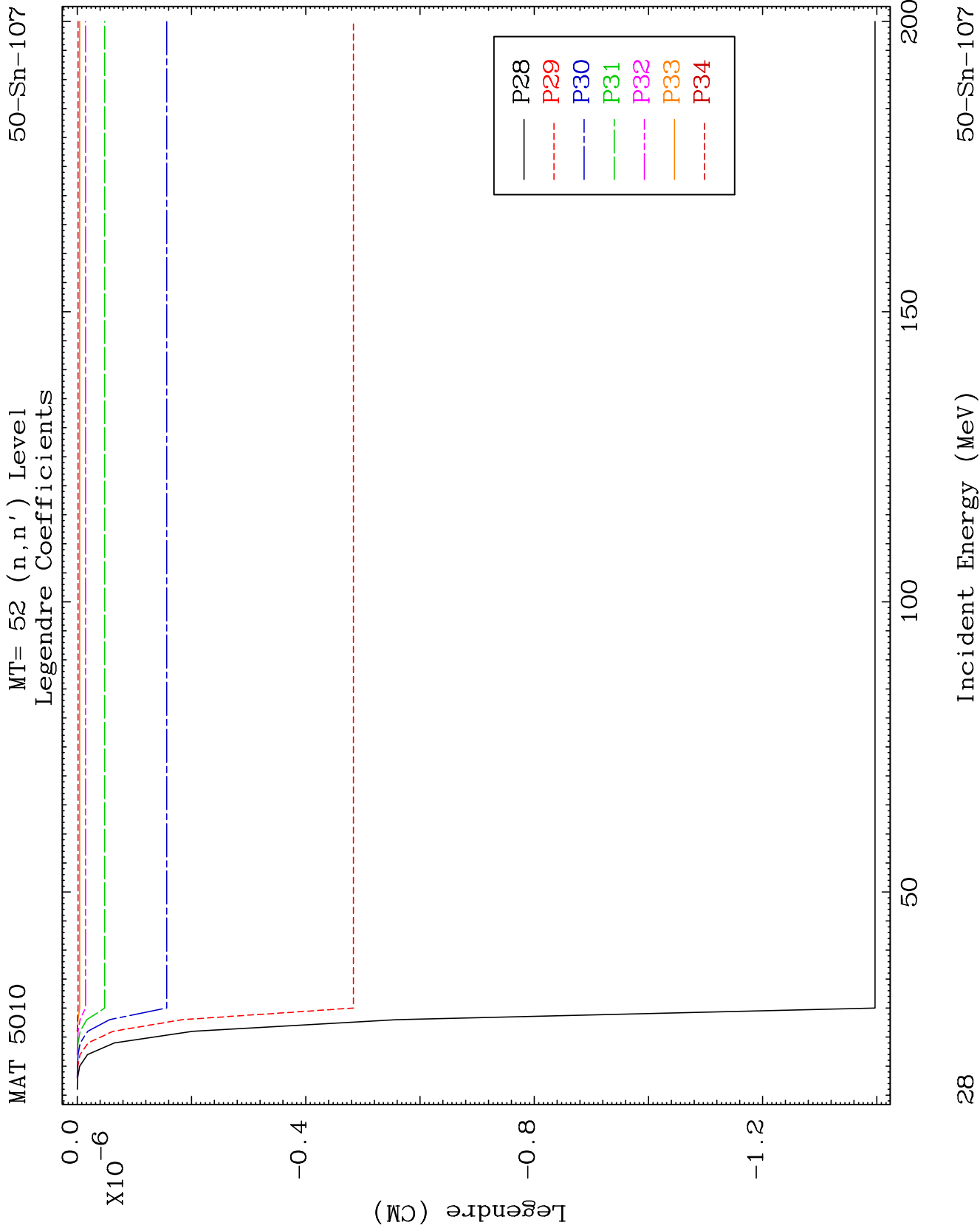
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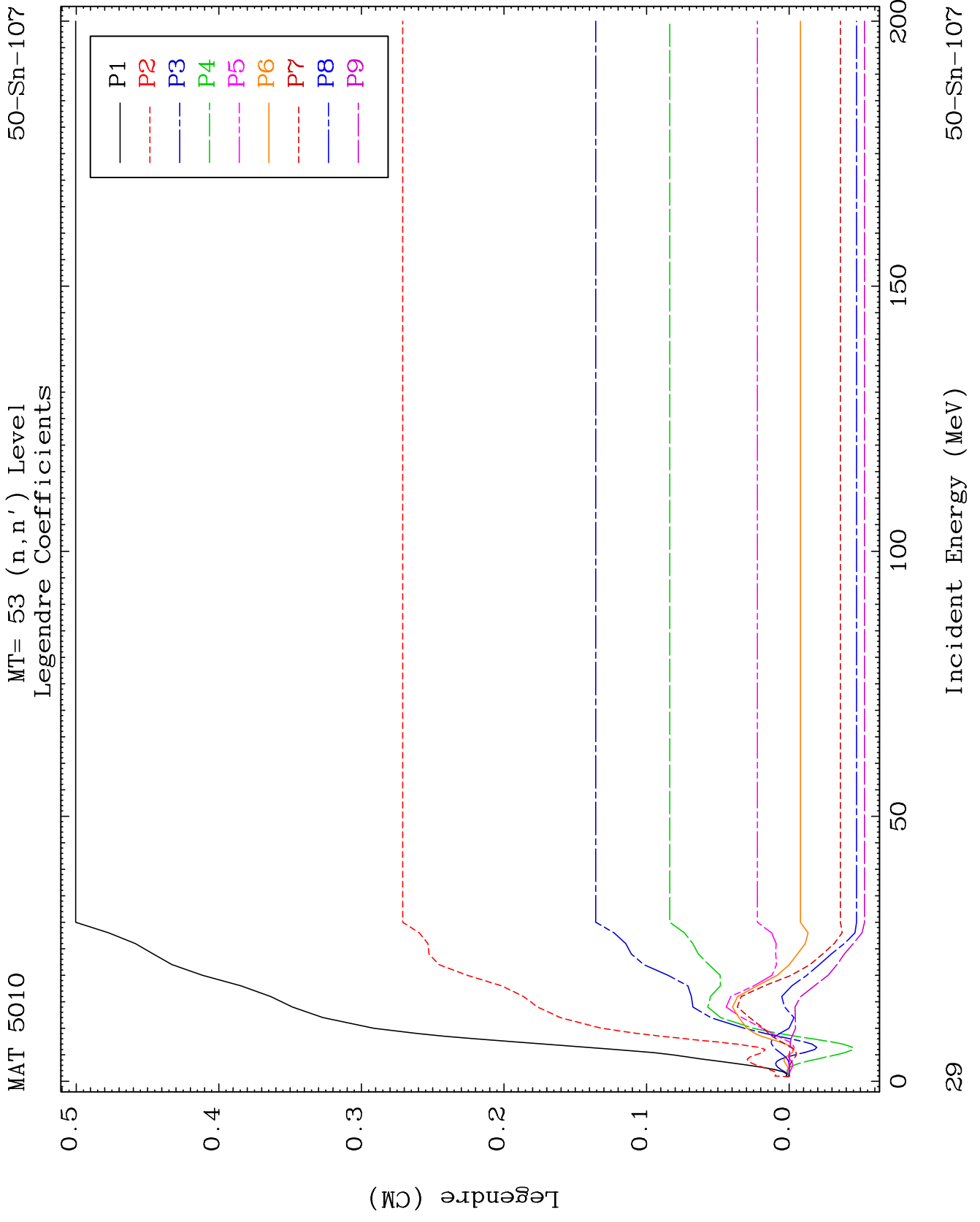


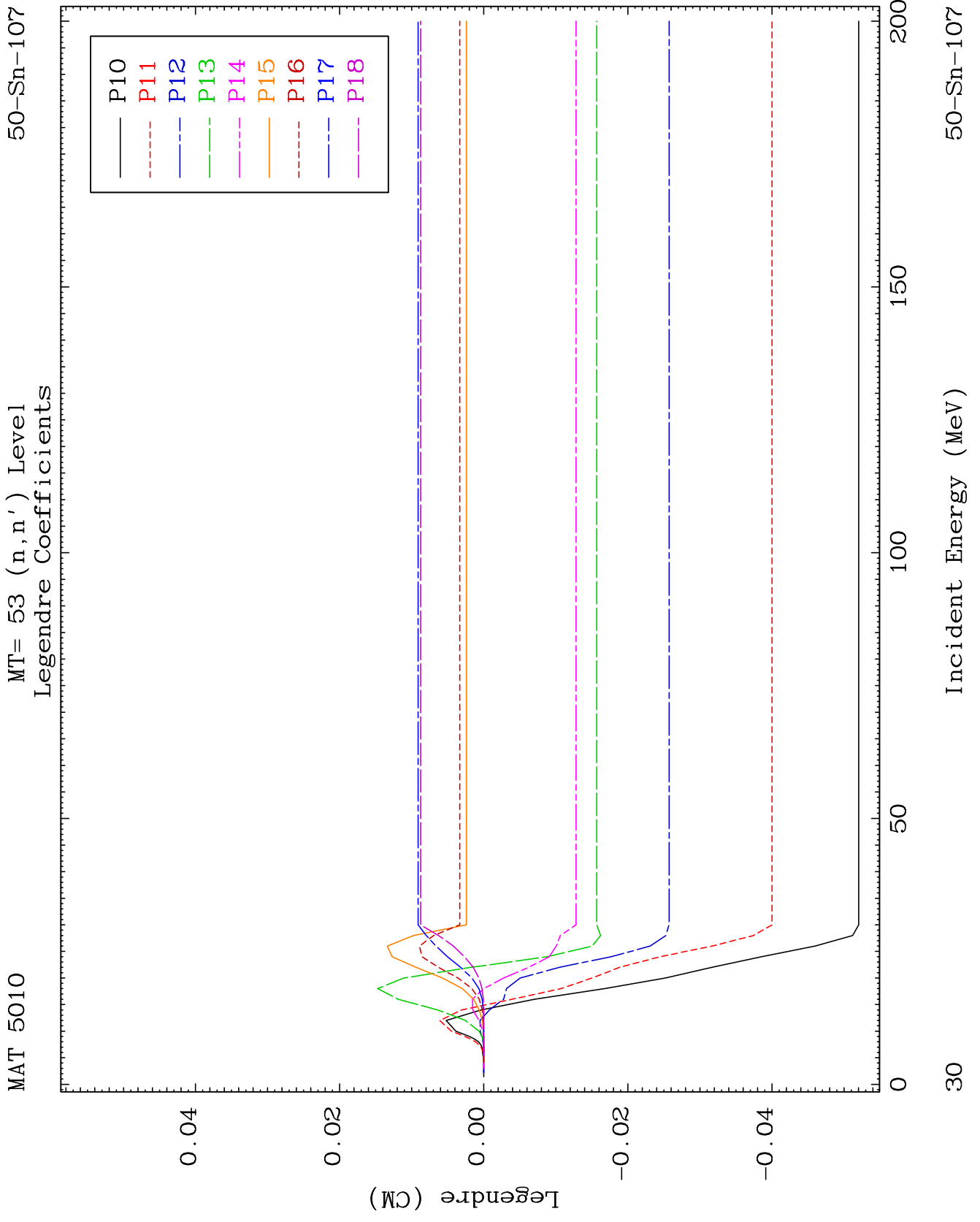


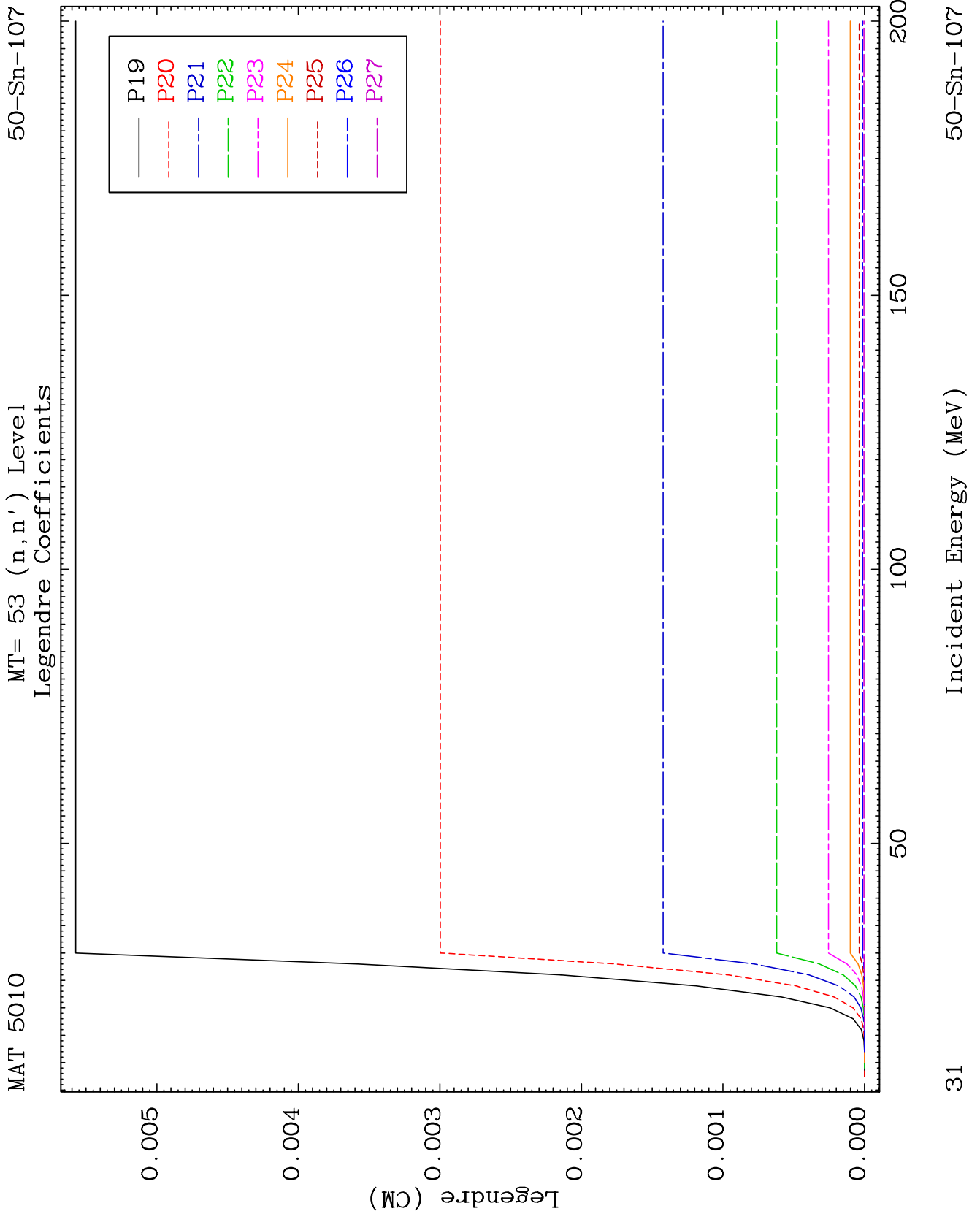


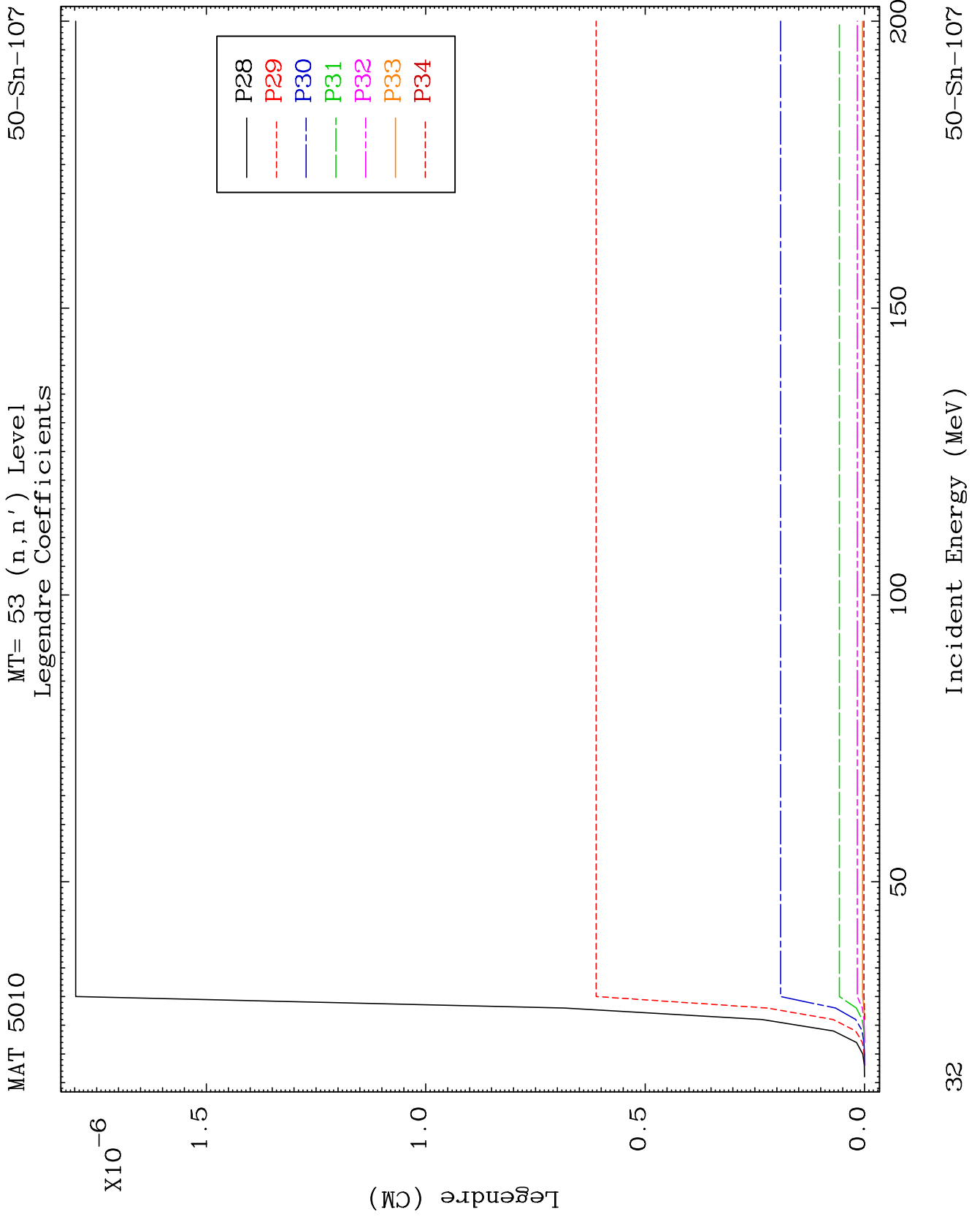


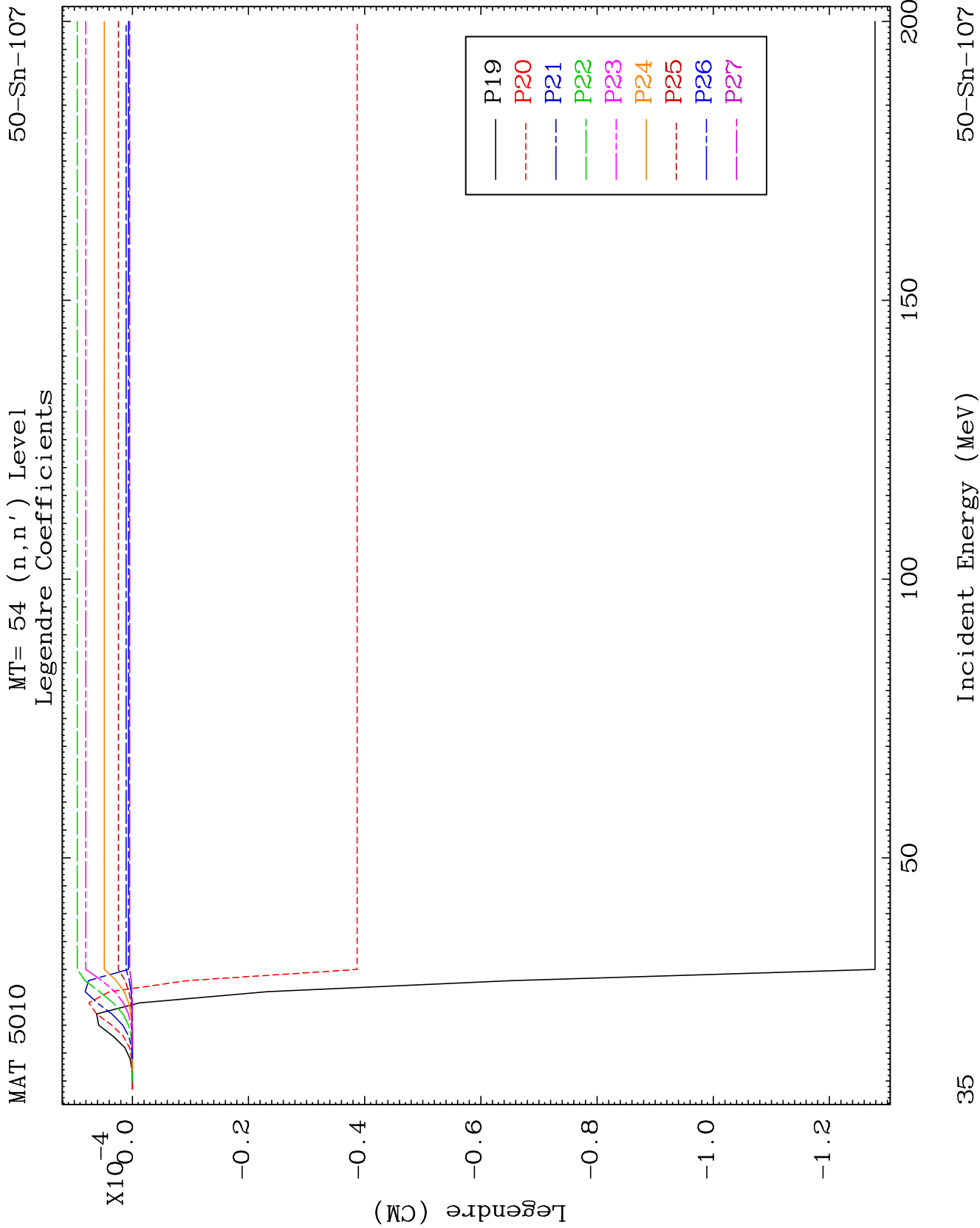


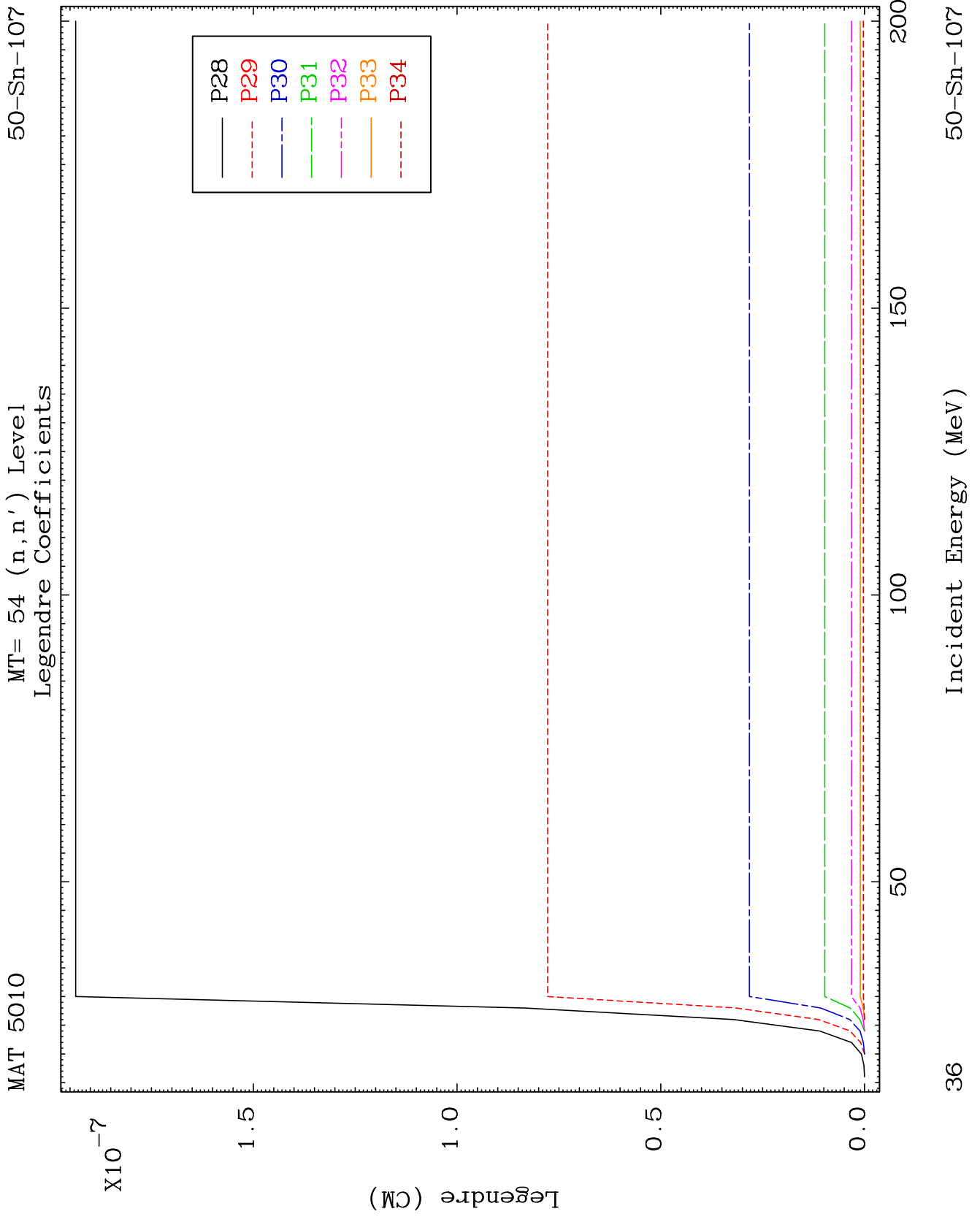


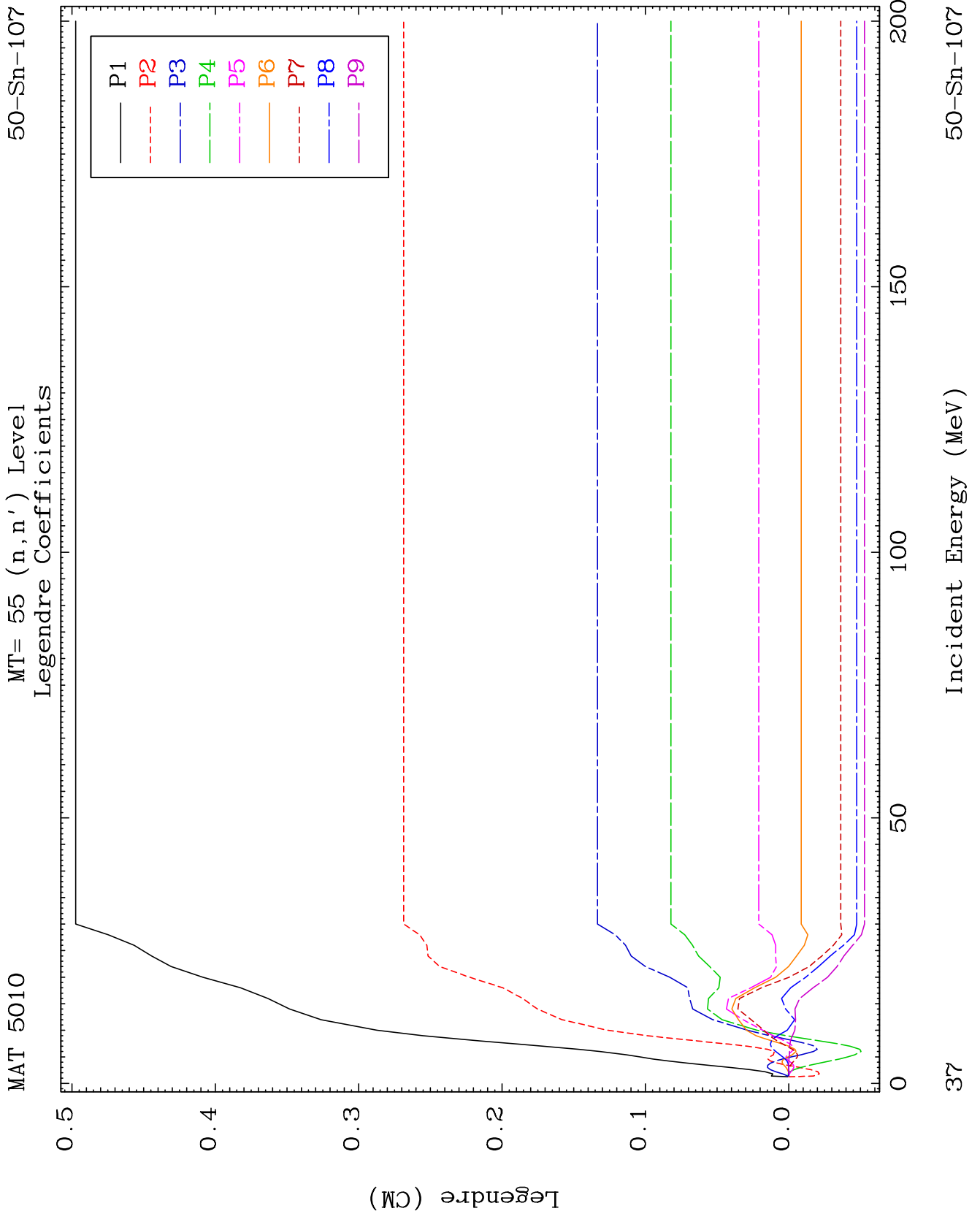


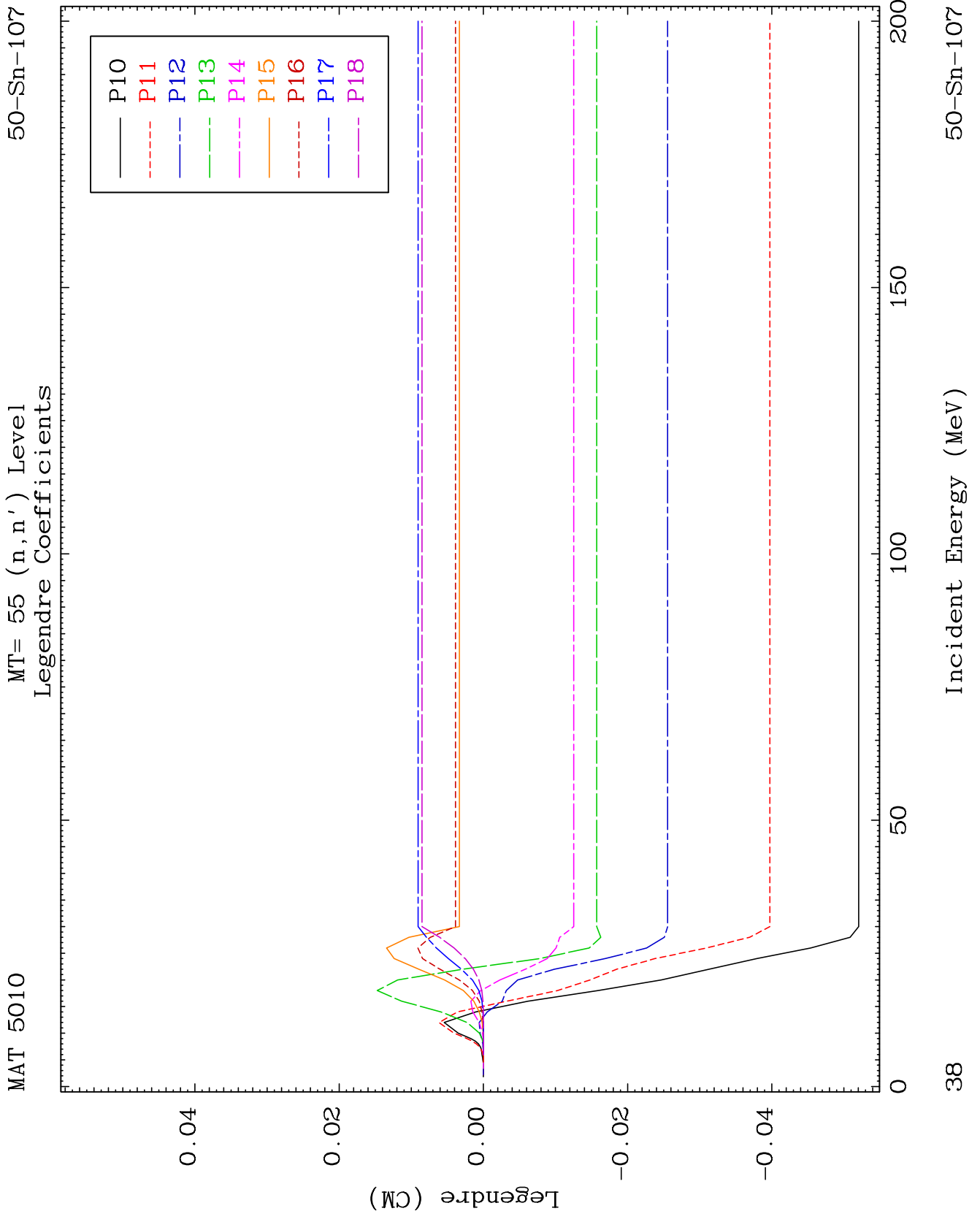


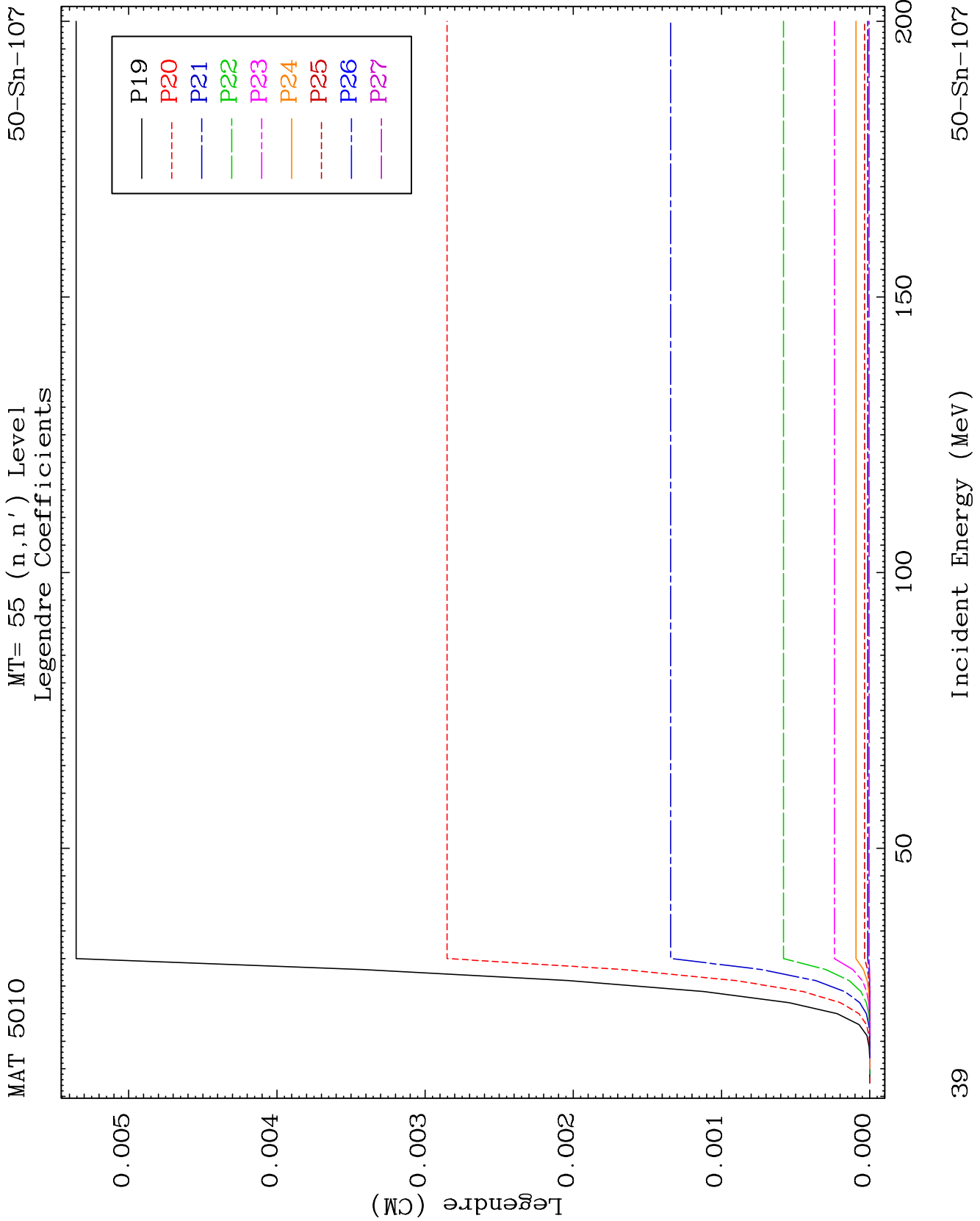


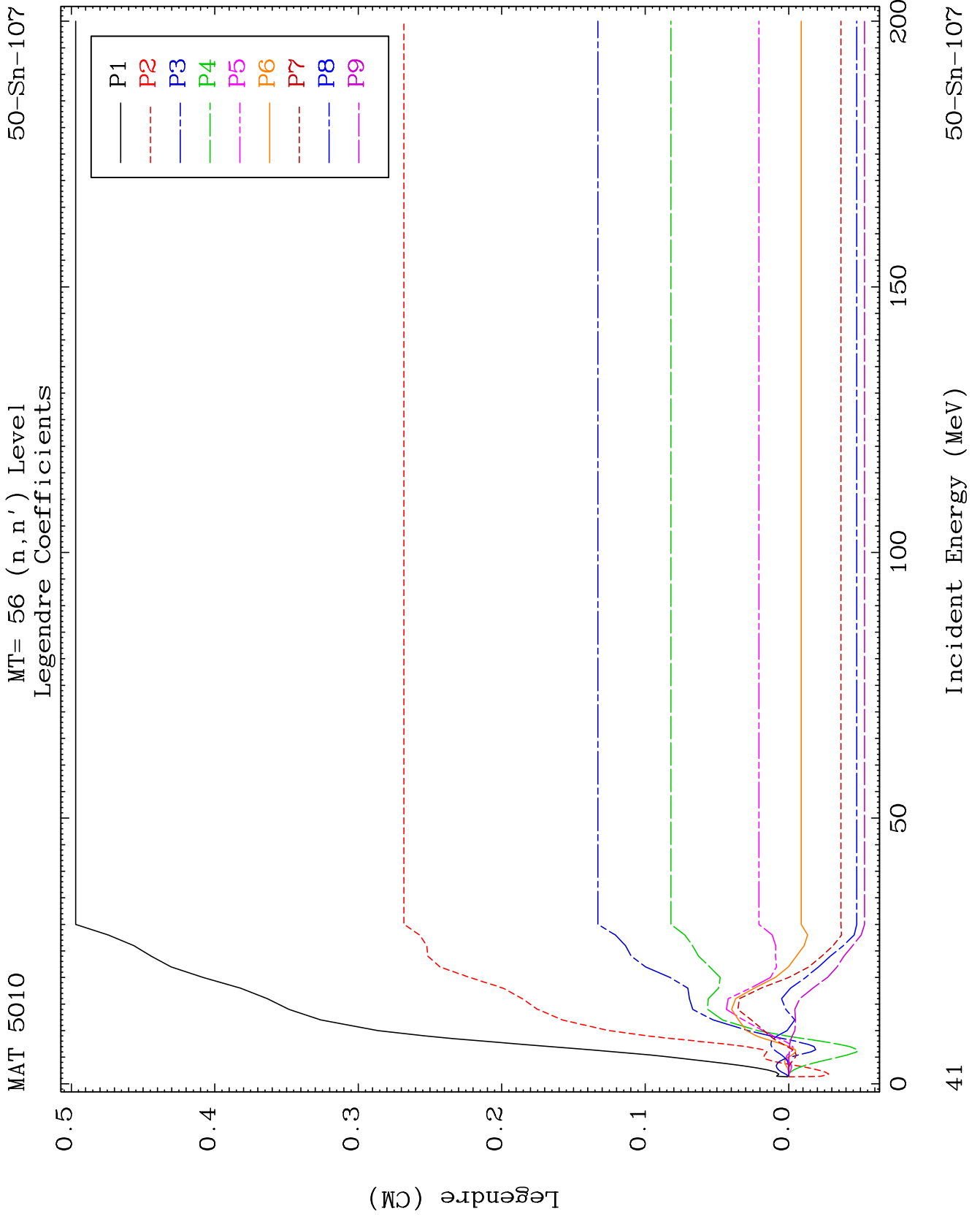


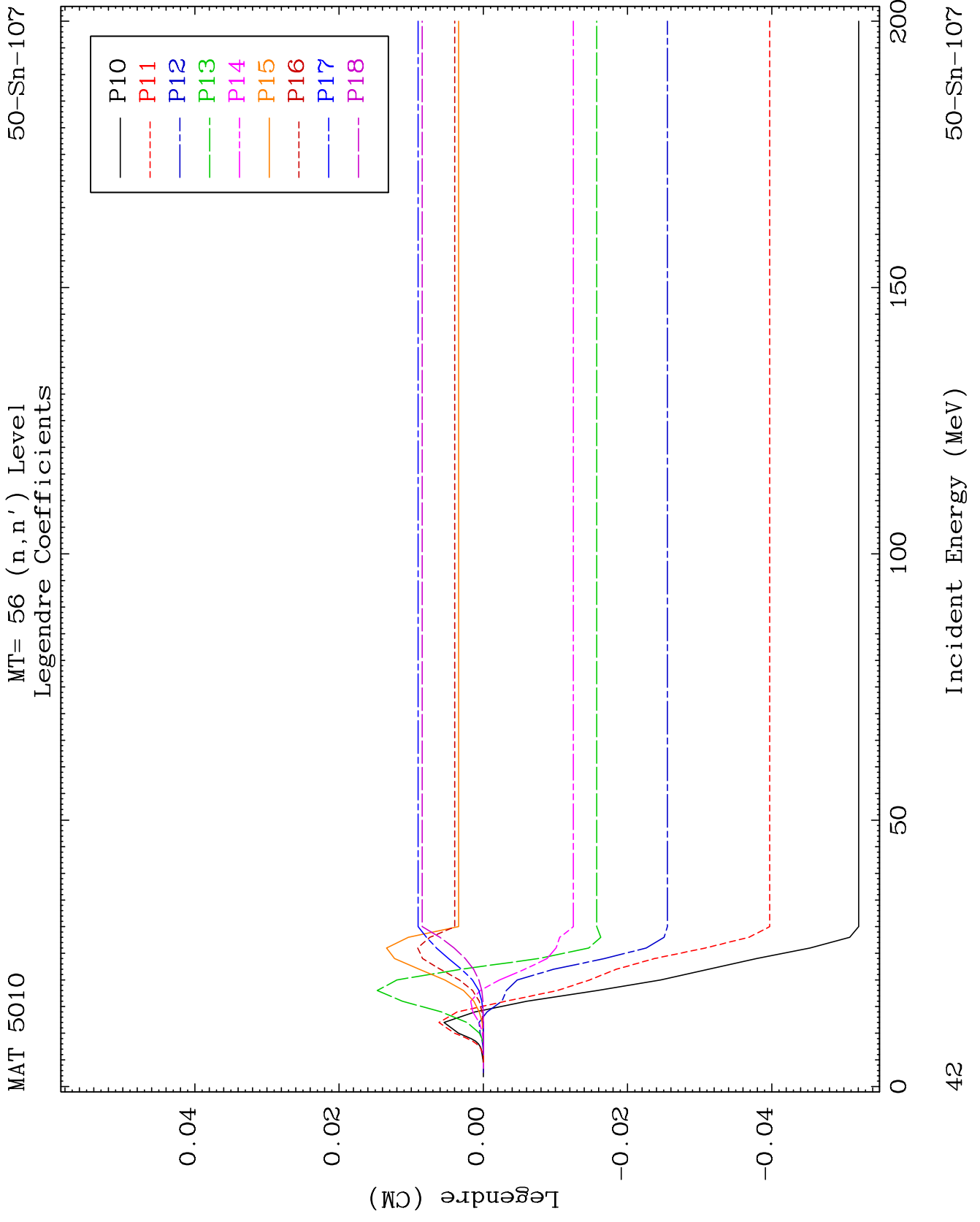


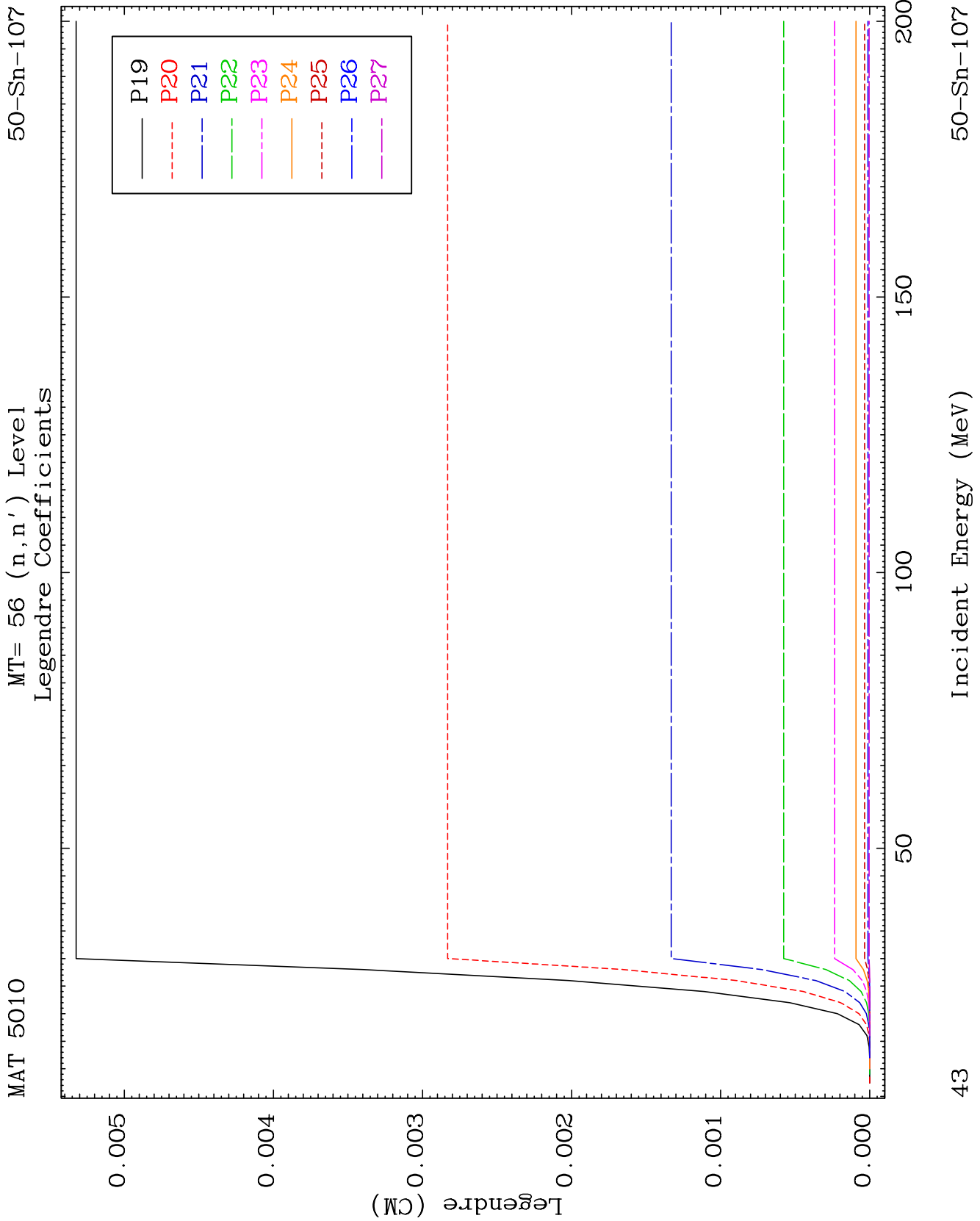


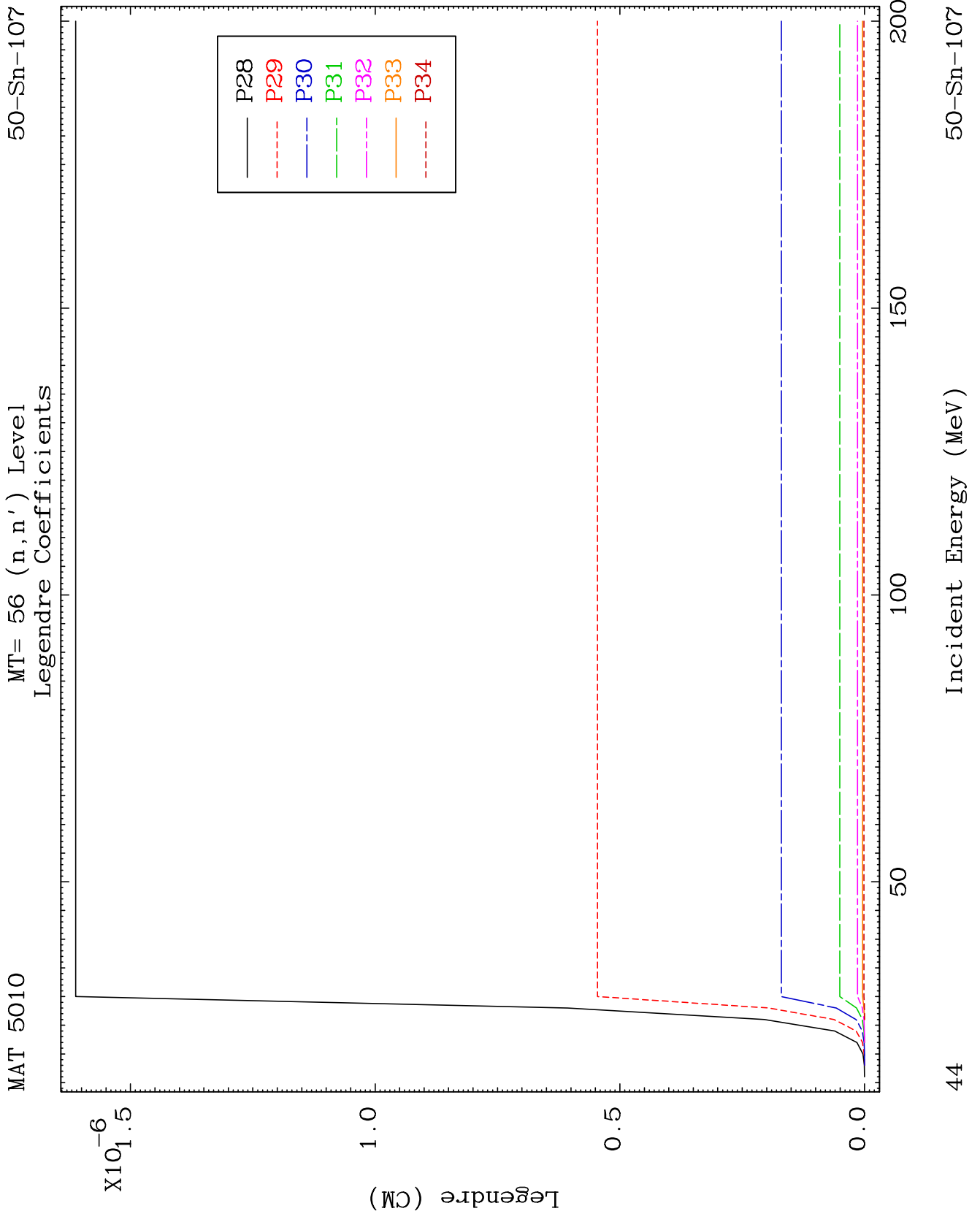


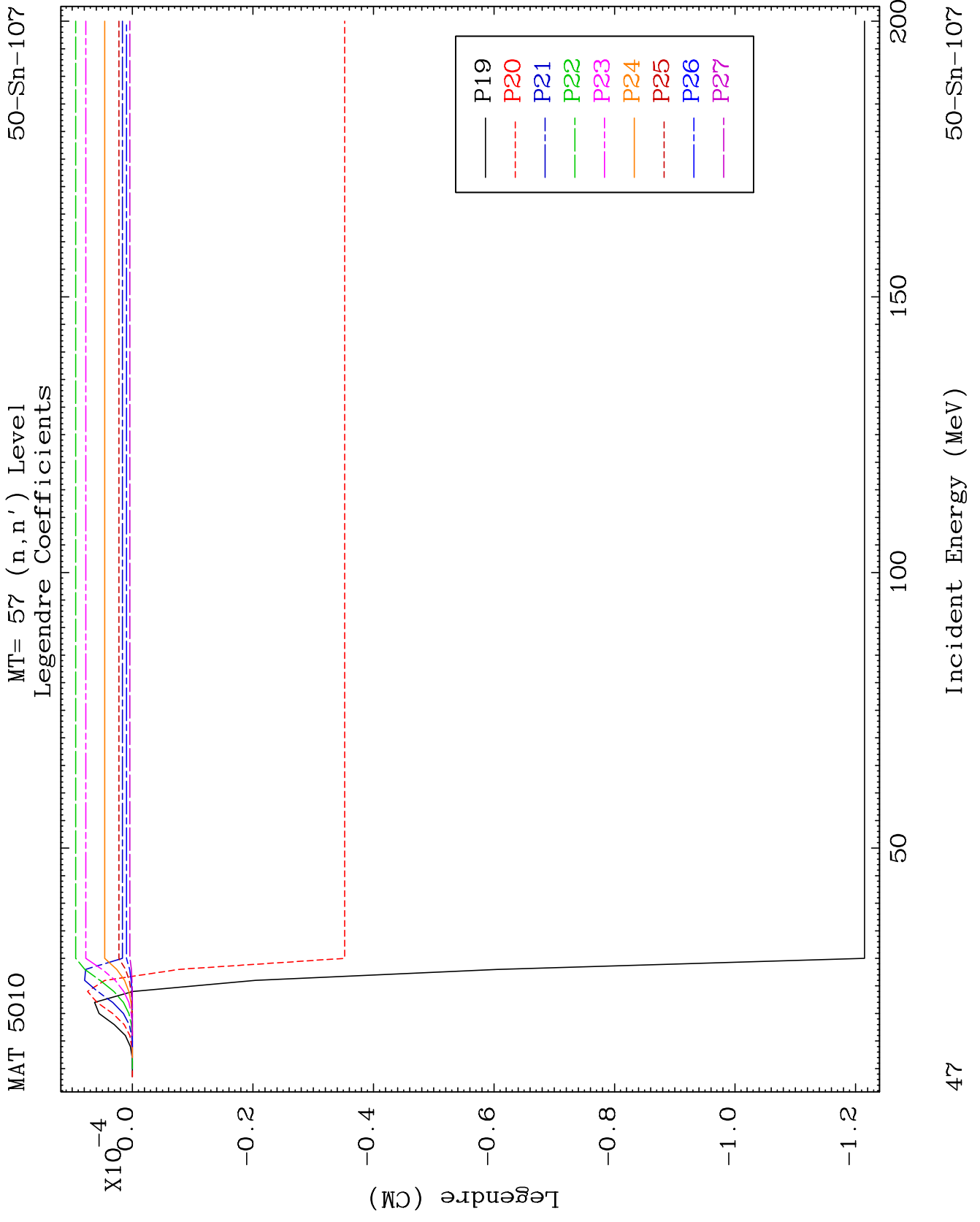


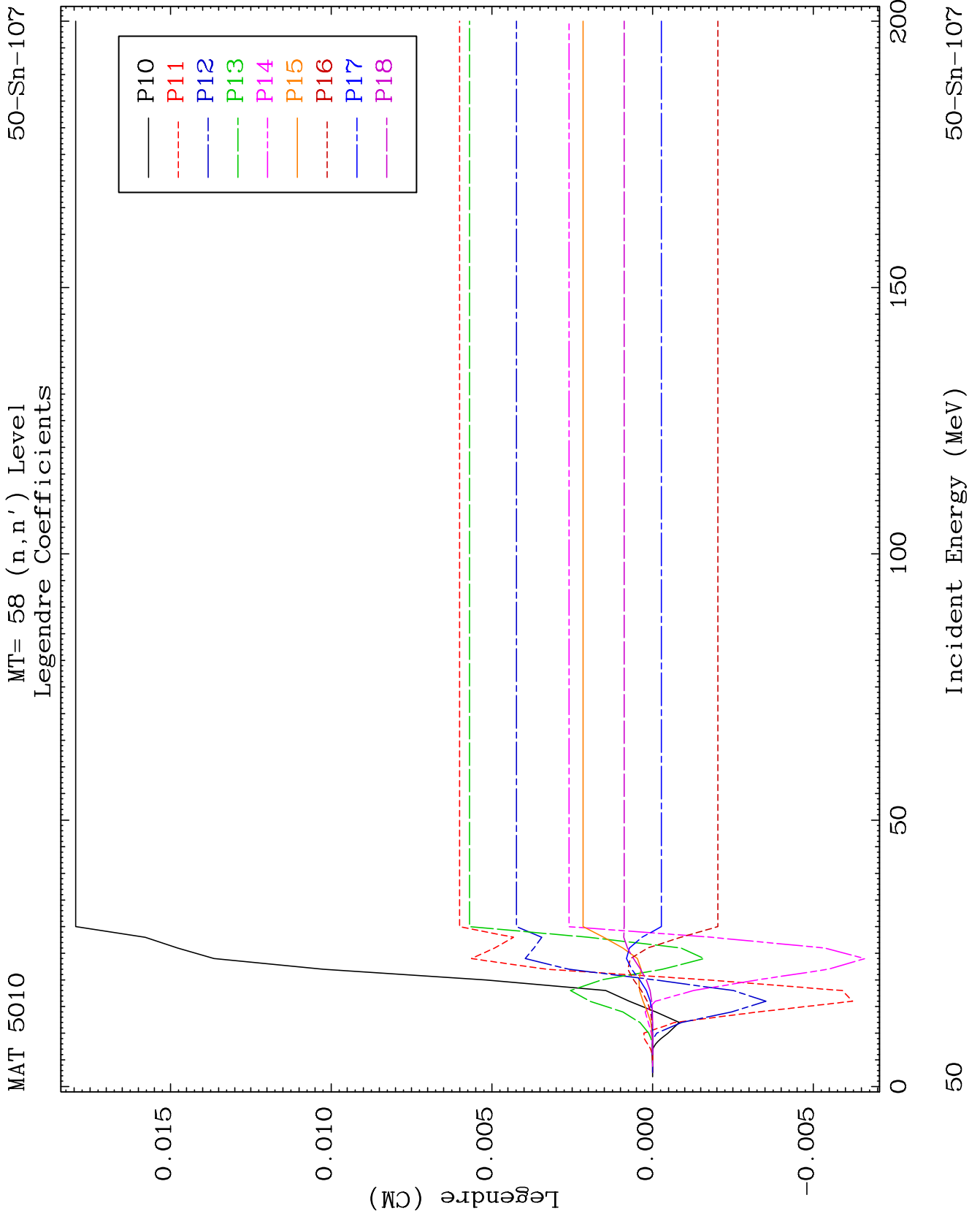


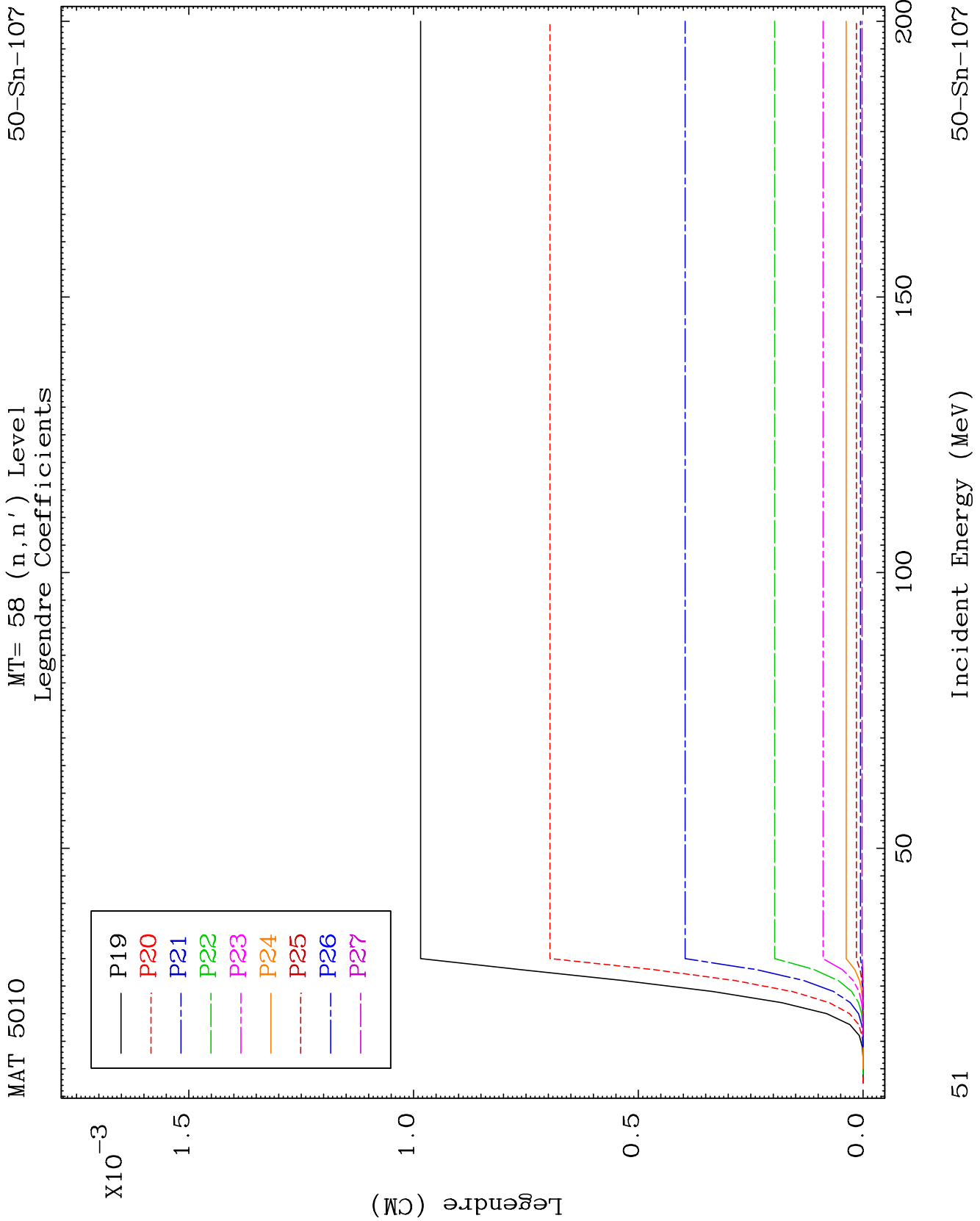


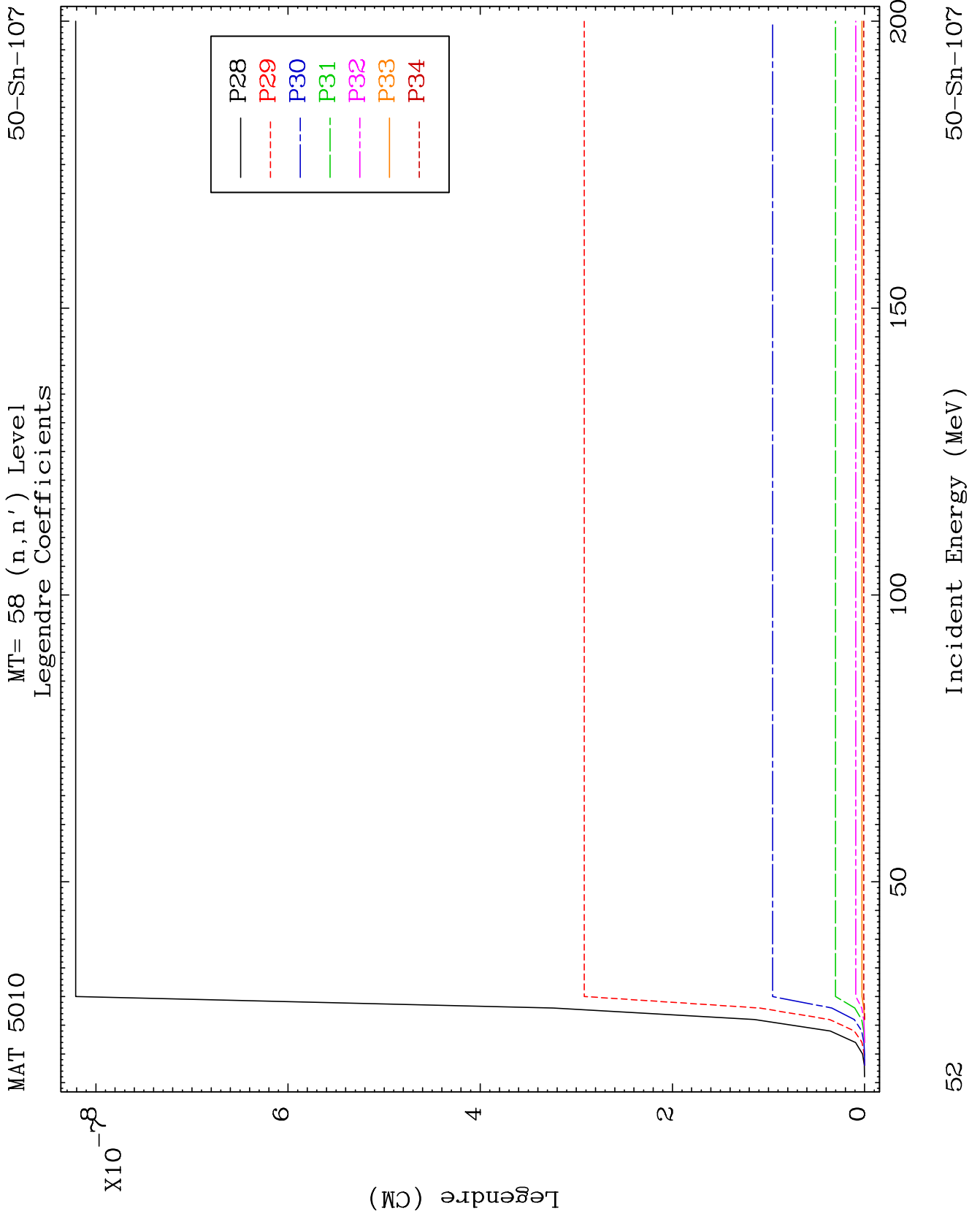


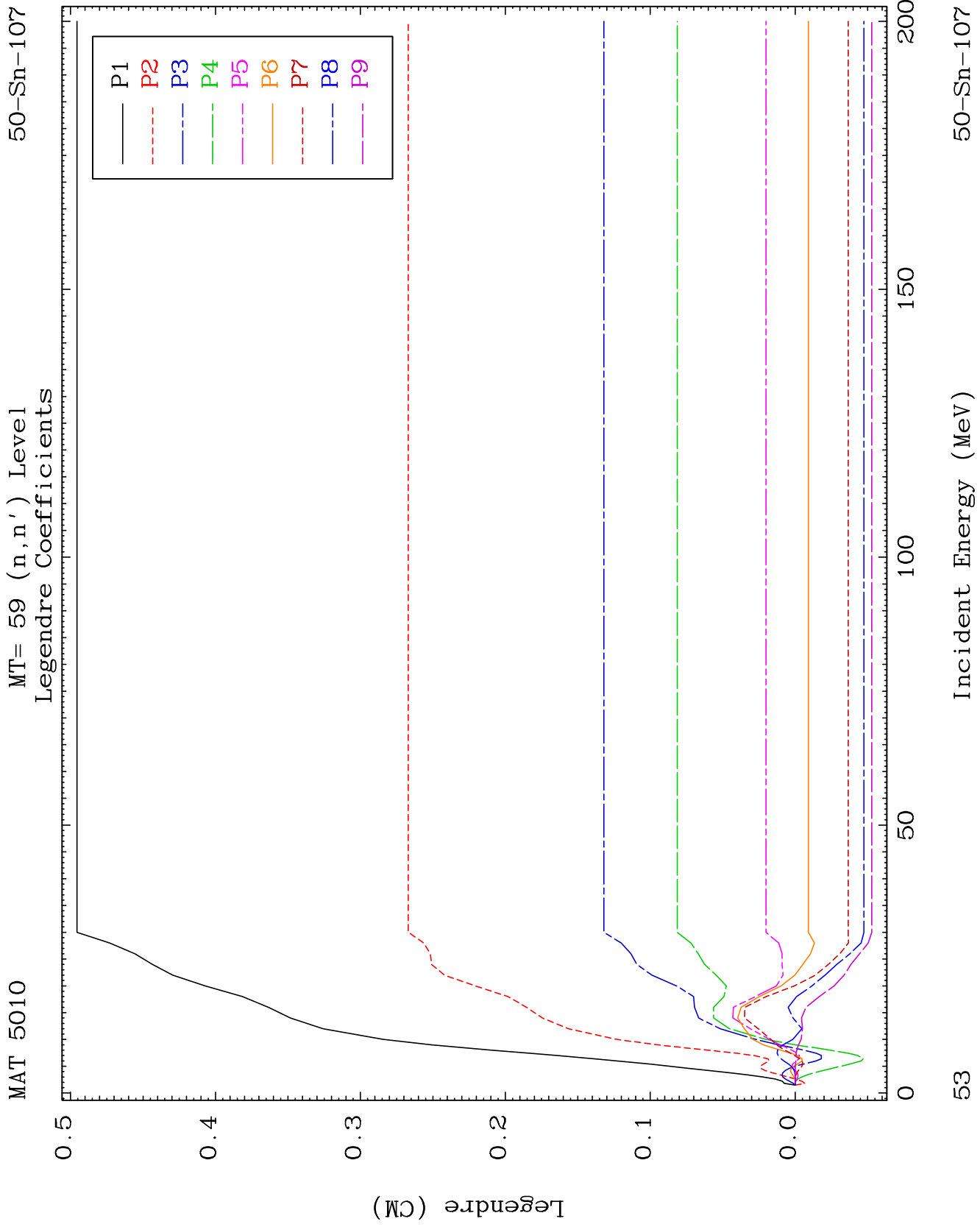


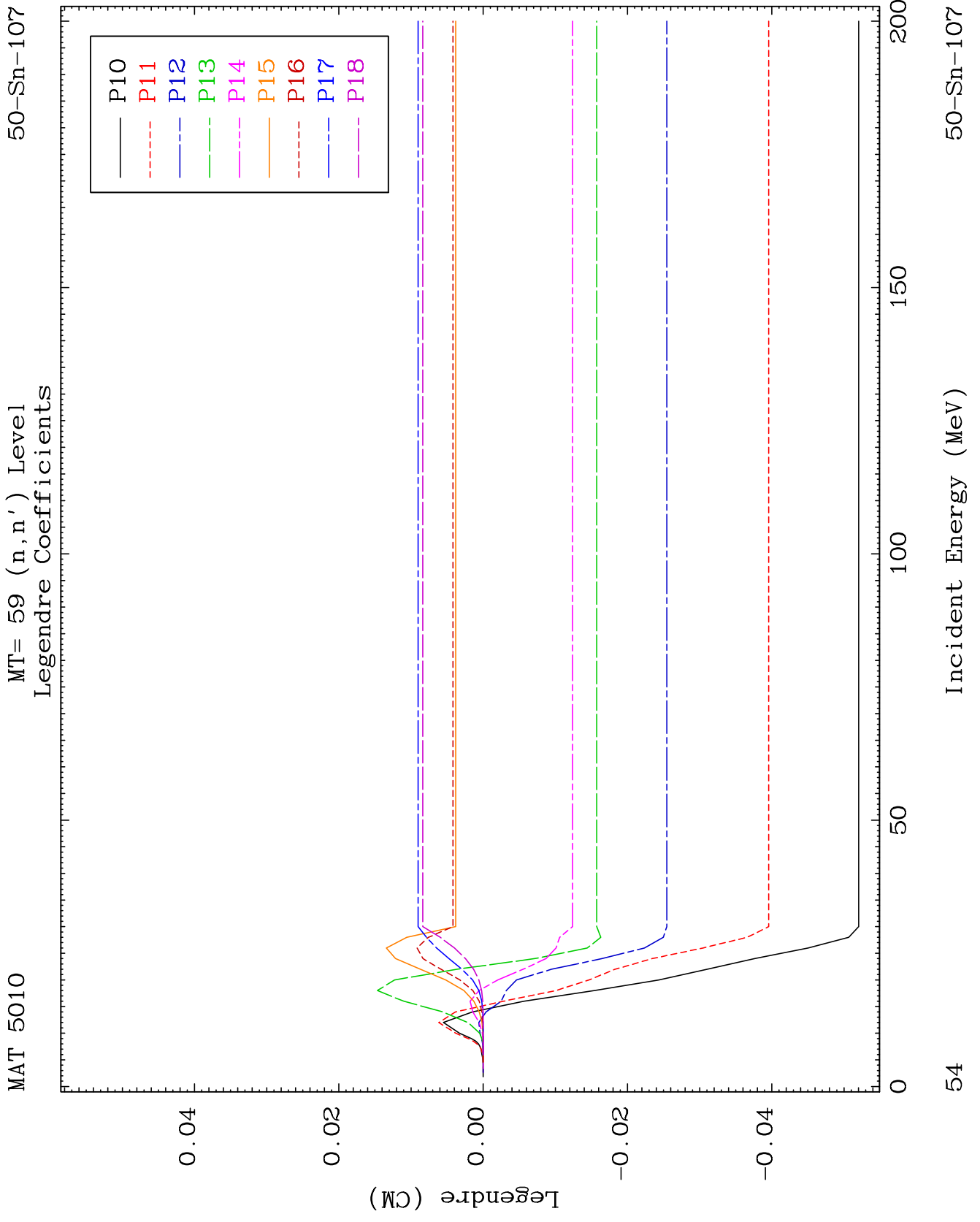


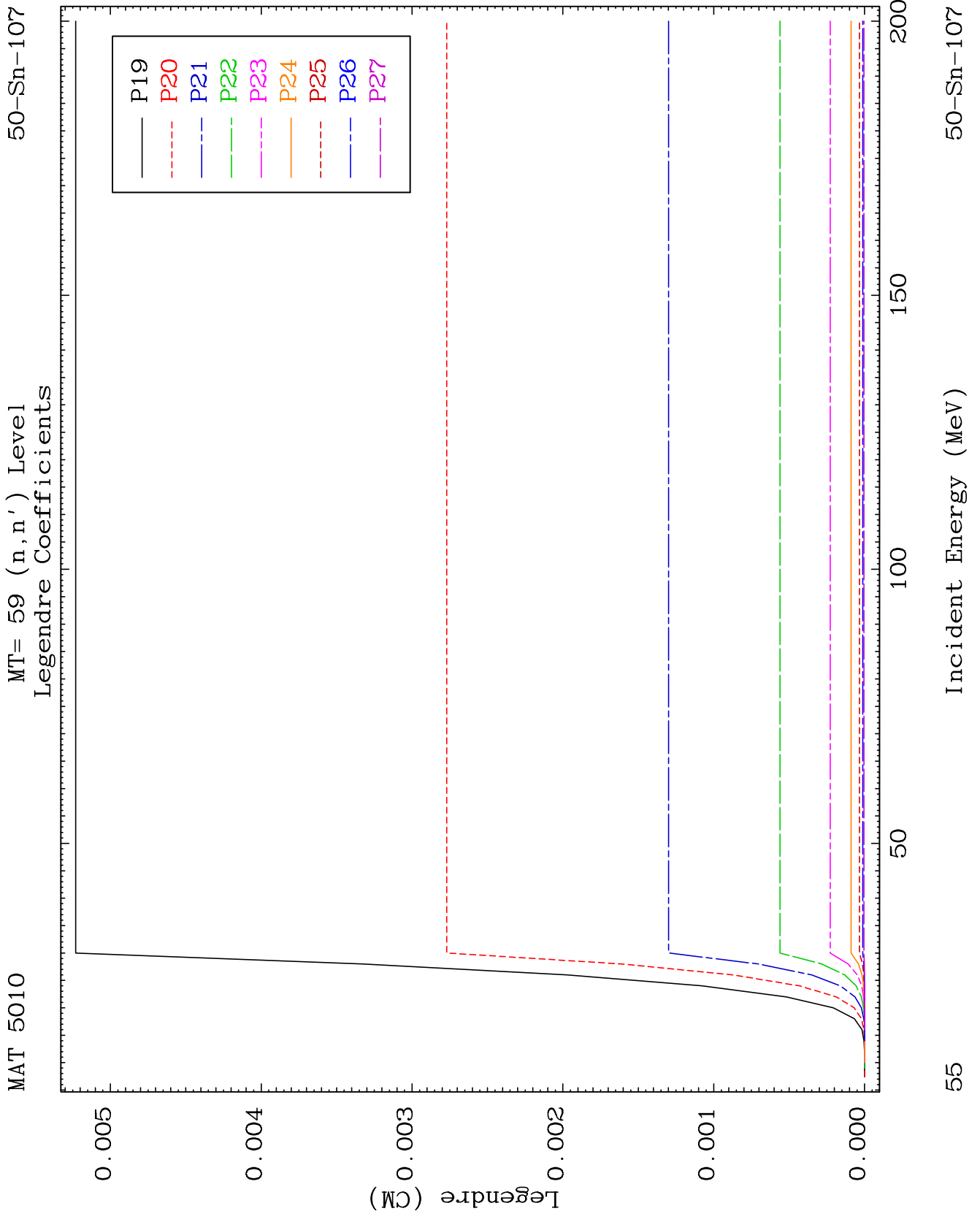


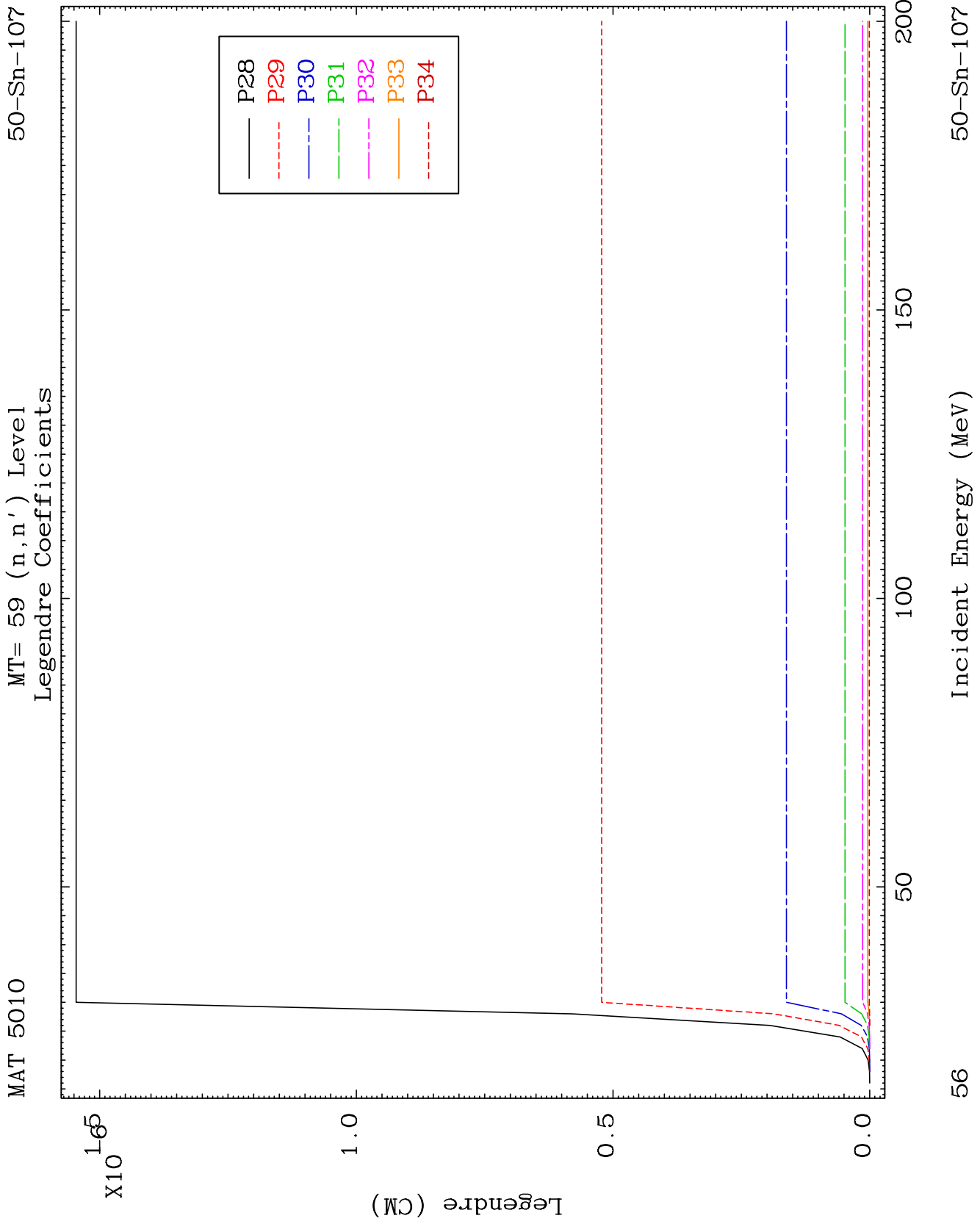


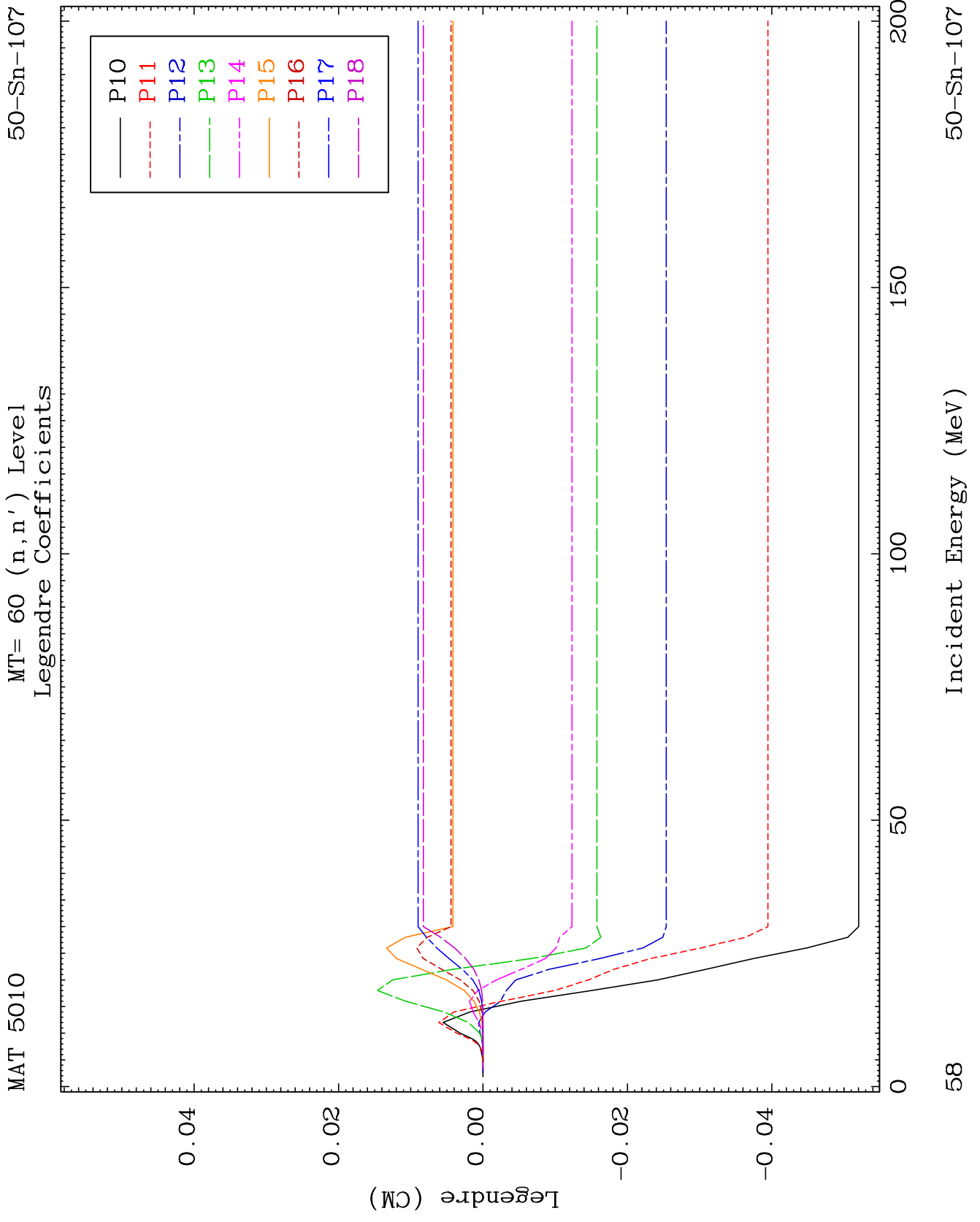


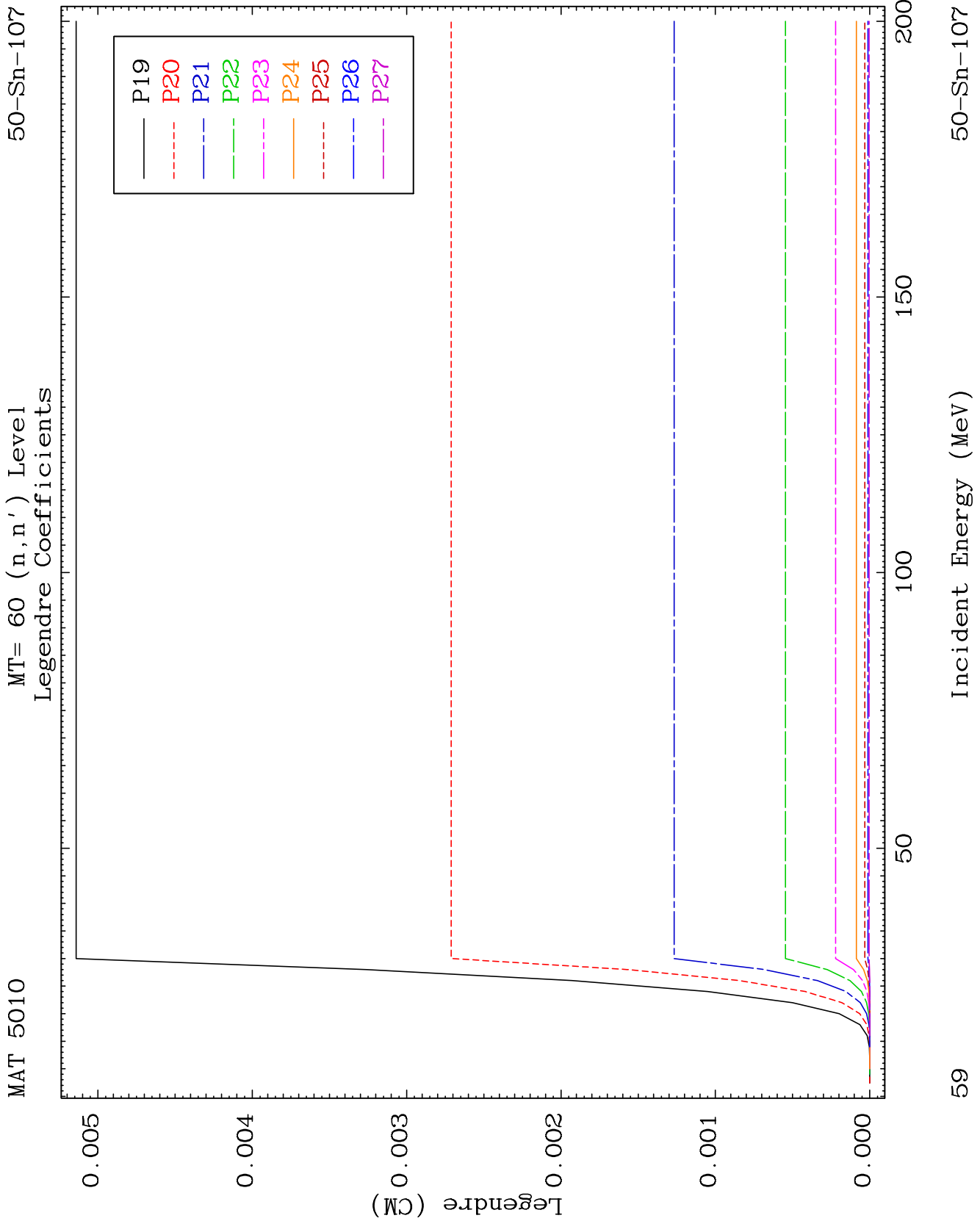


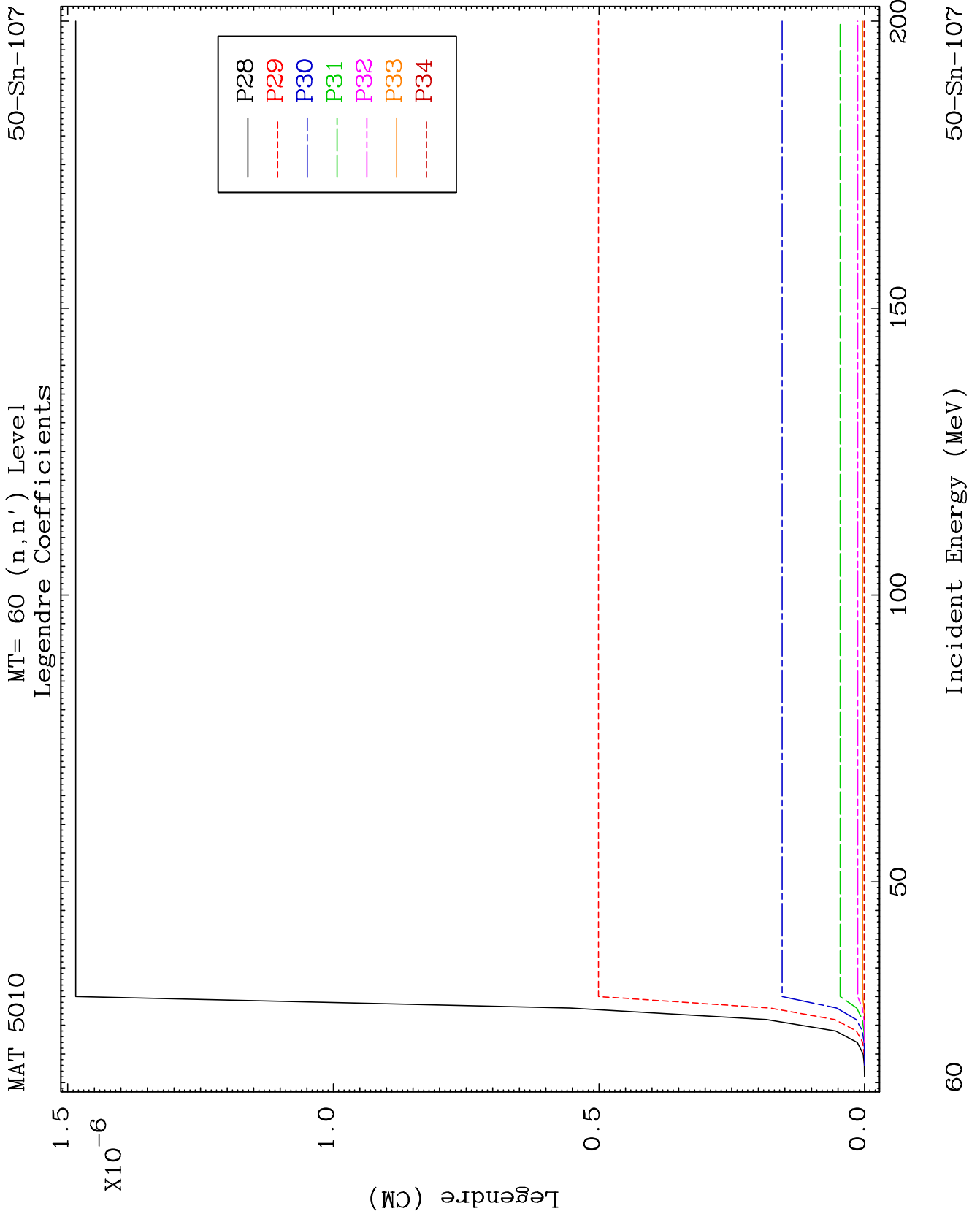


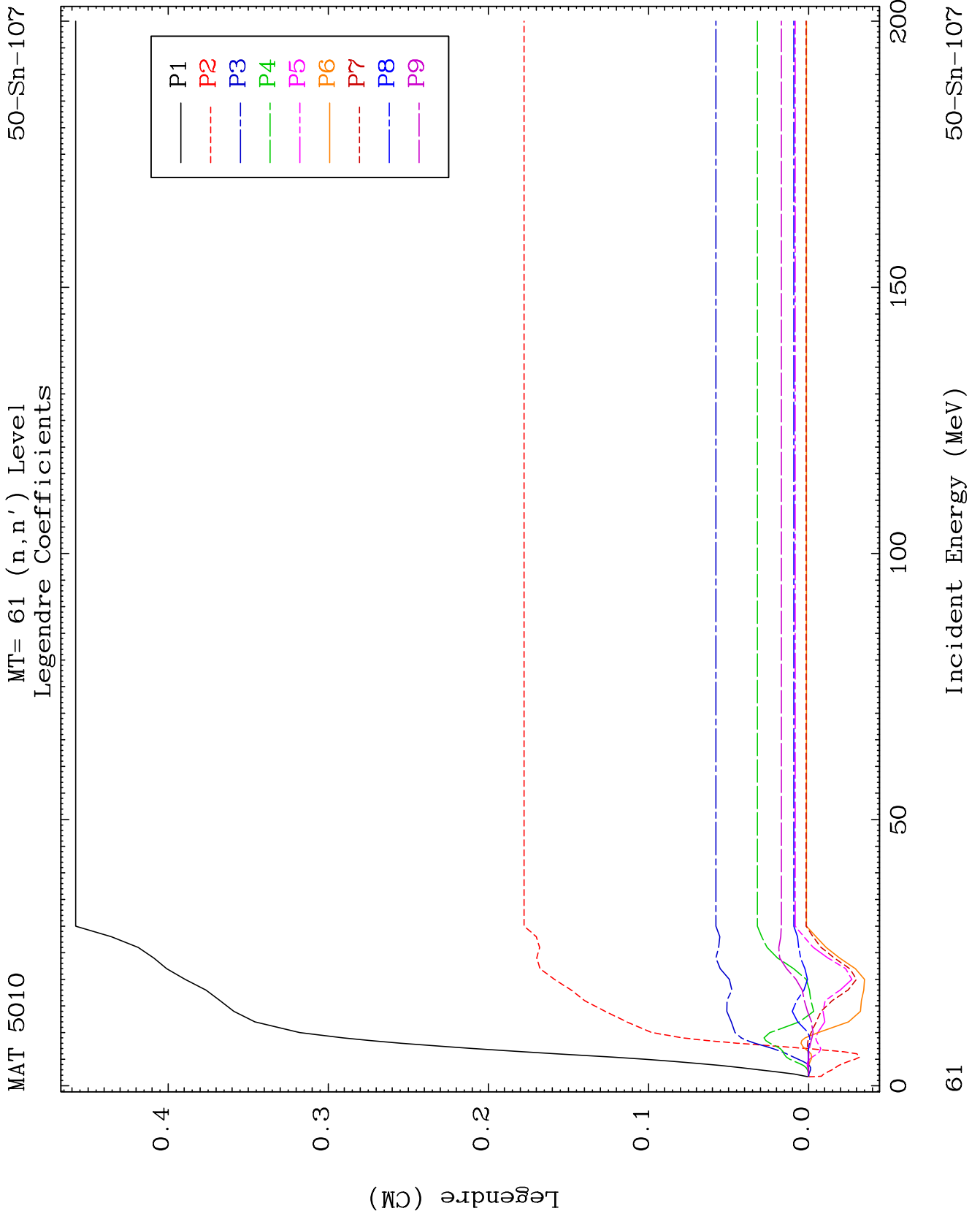


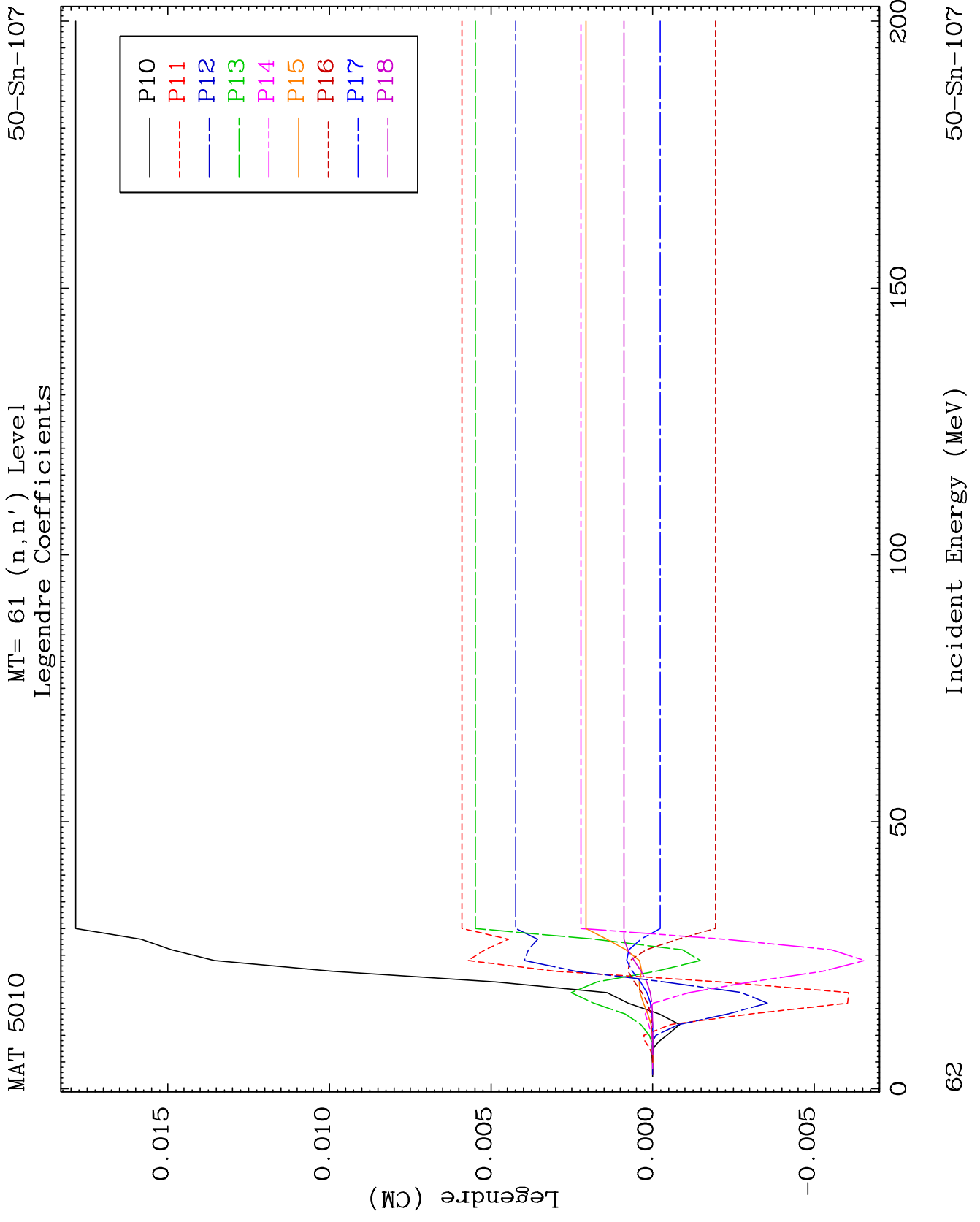


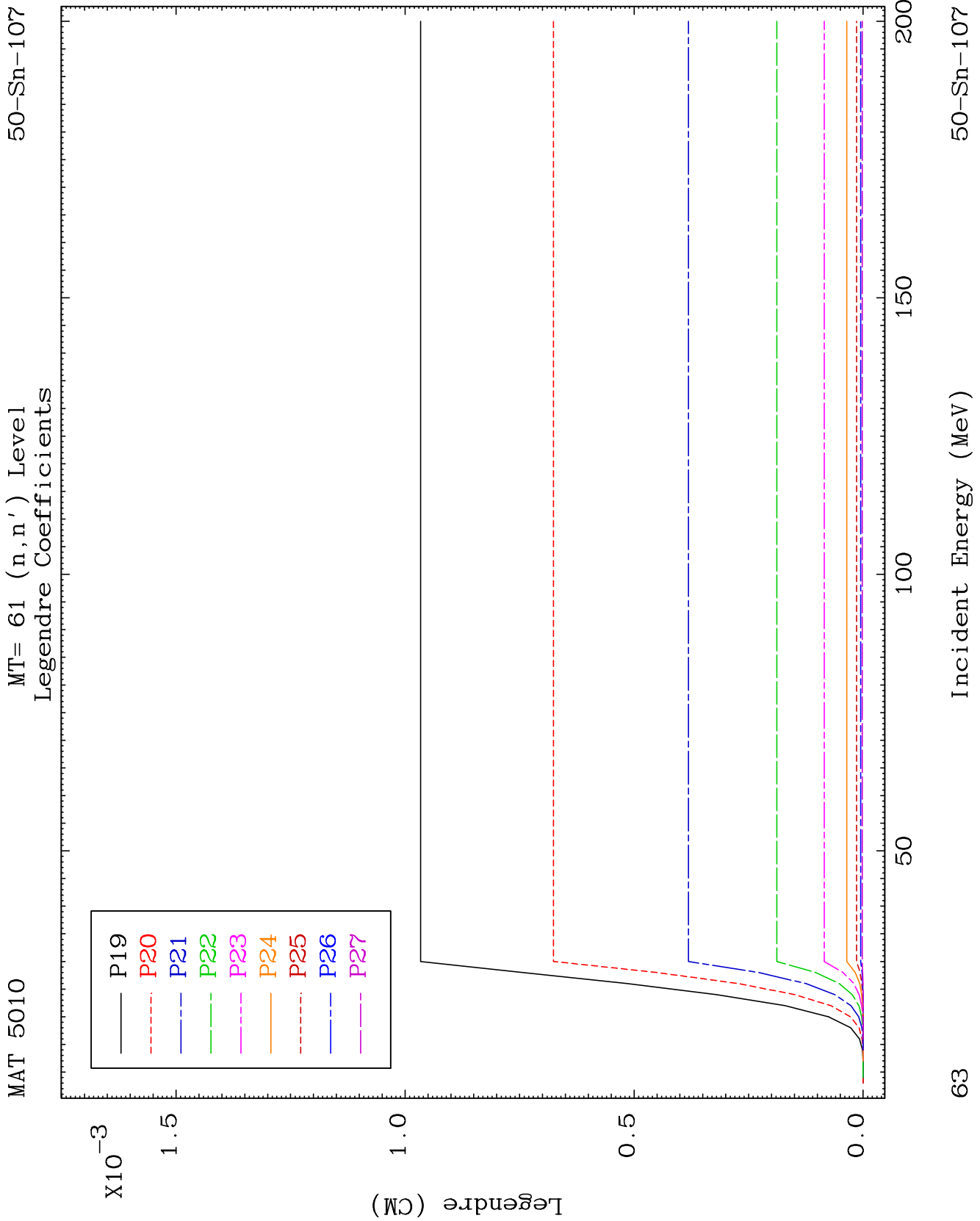








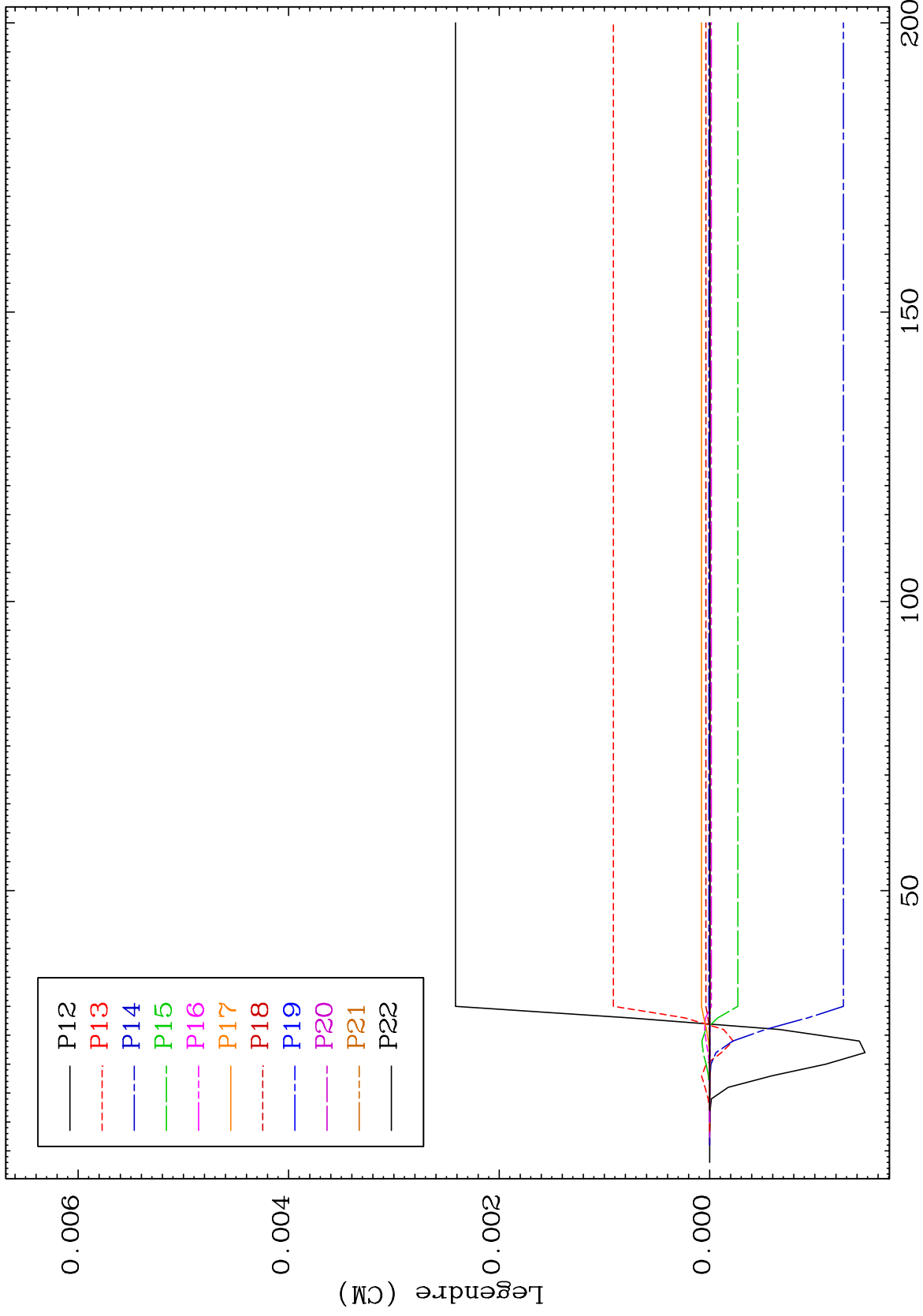


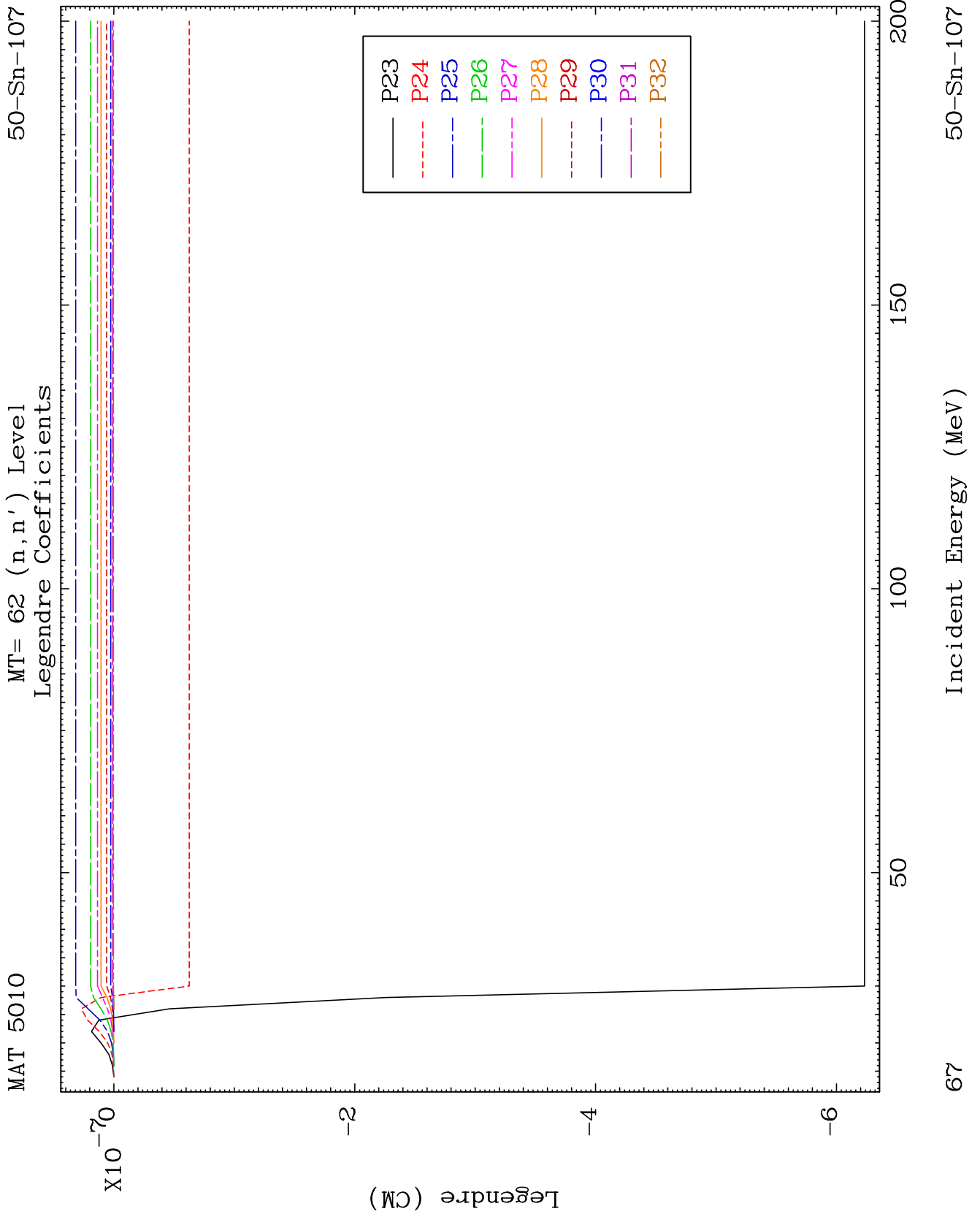


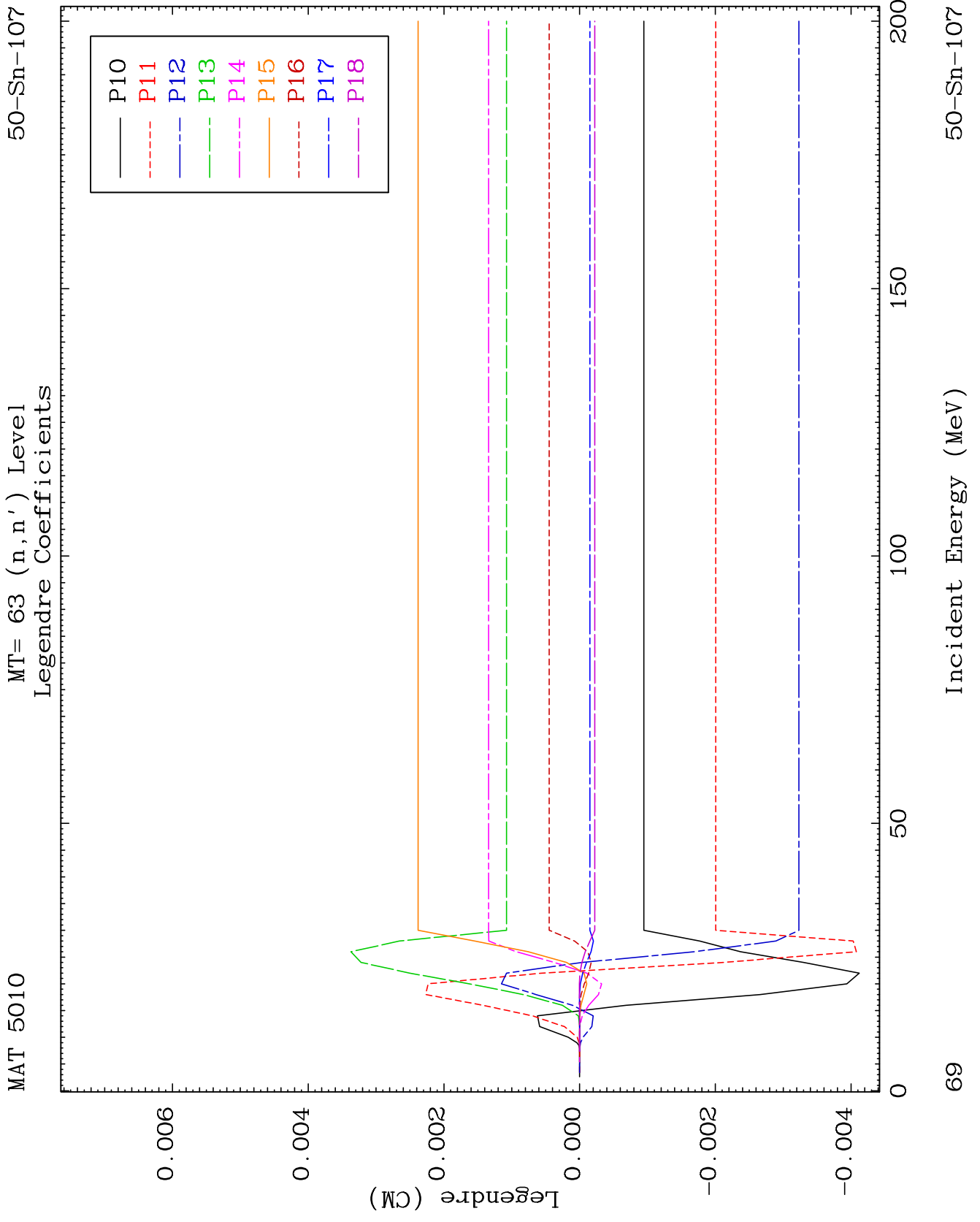
MAT 5010

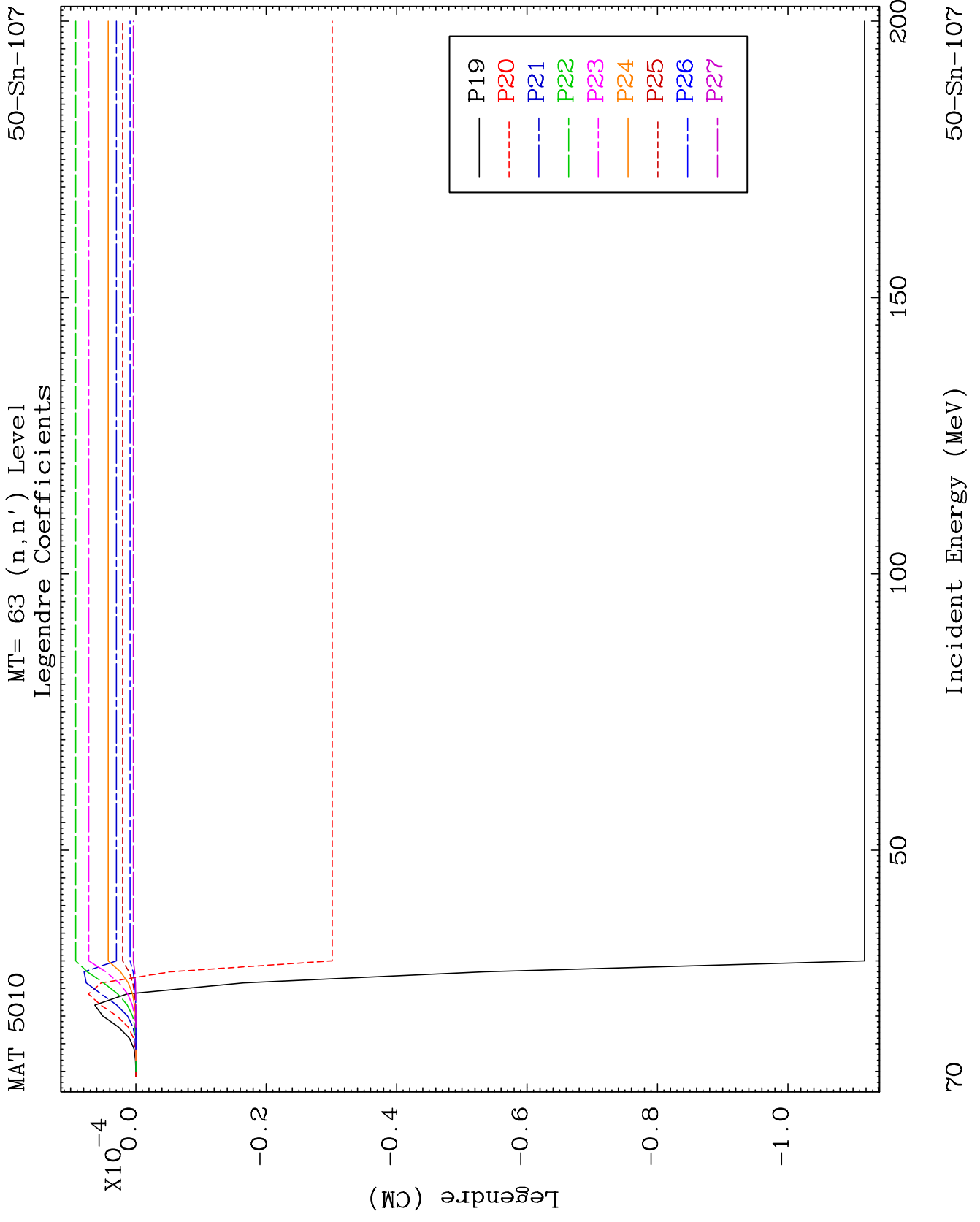
MT= 62 (n,n') Level
Legendre Coefficients

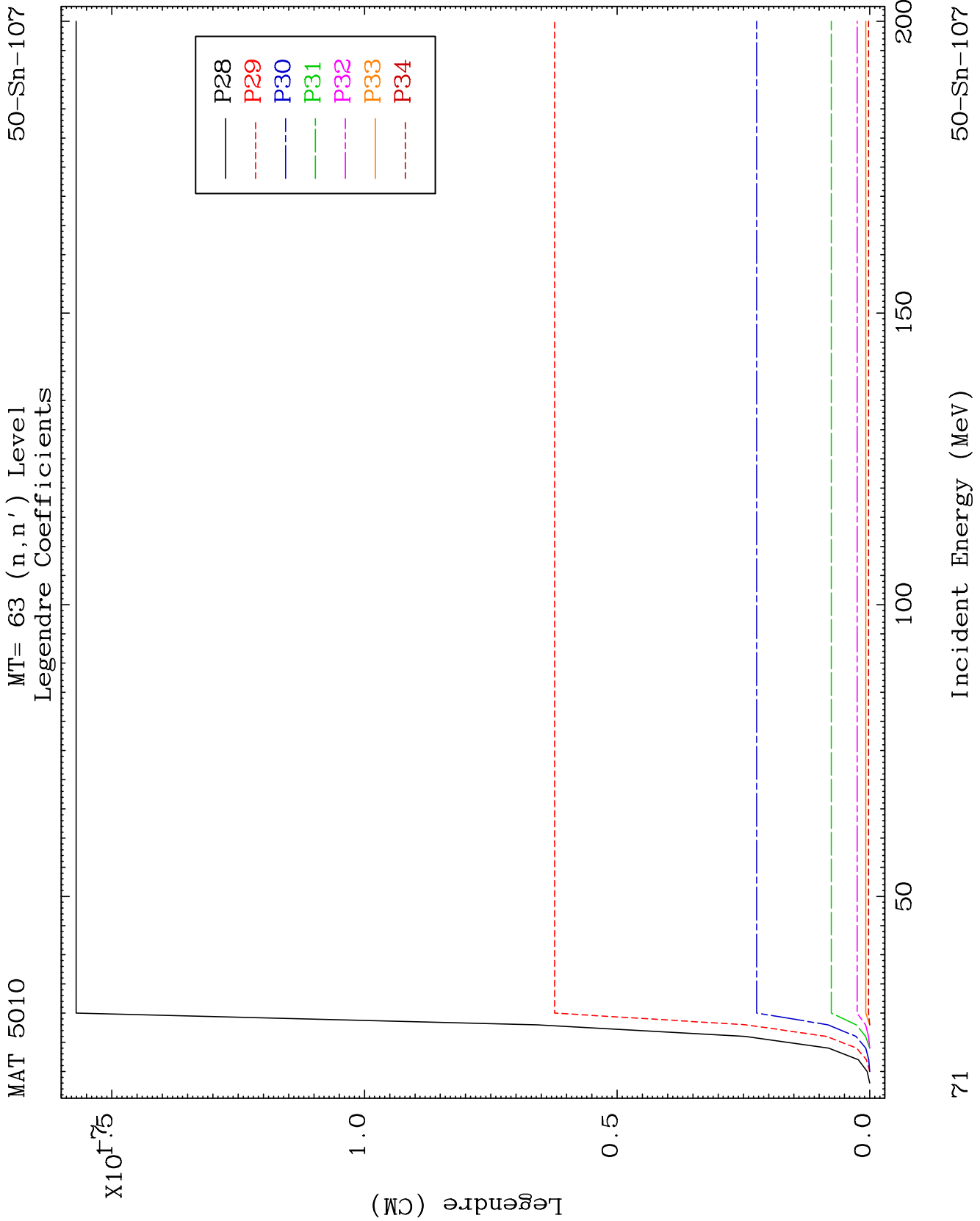
50-Sn-107

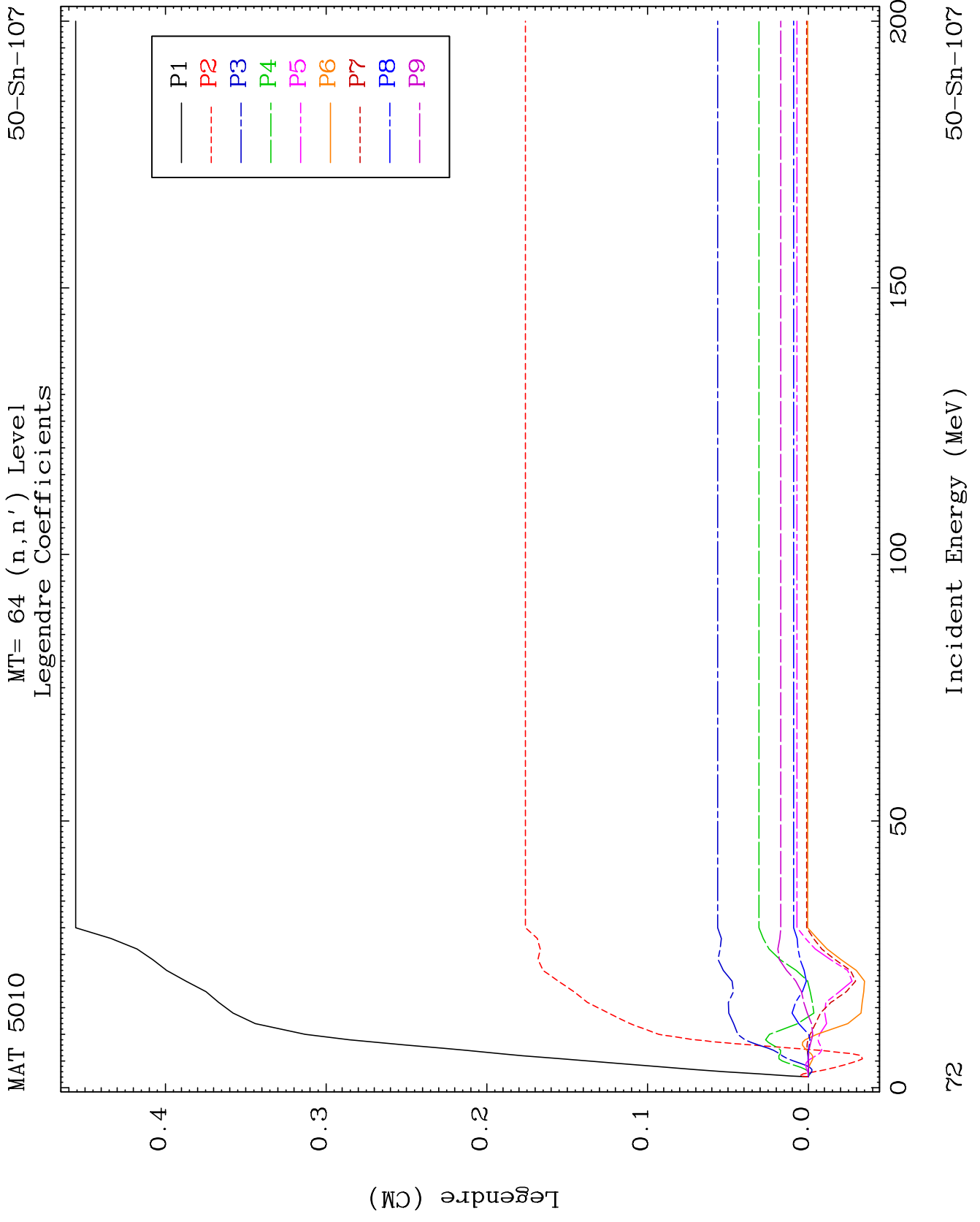


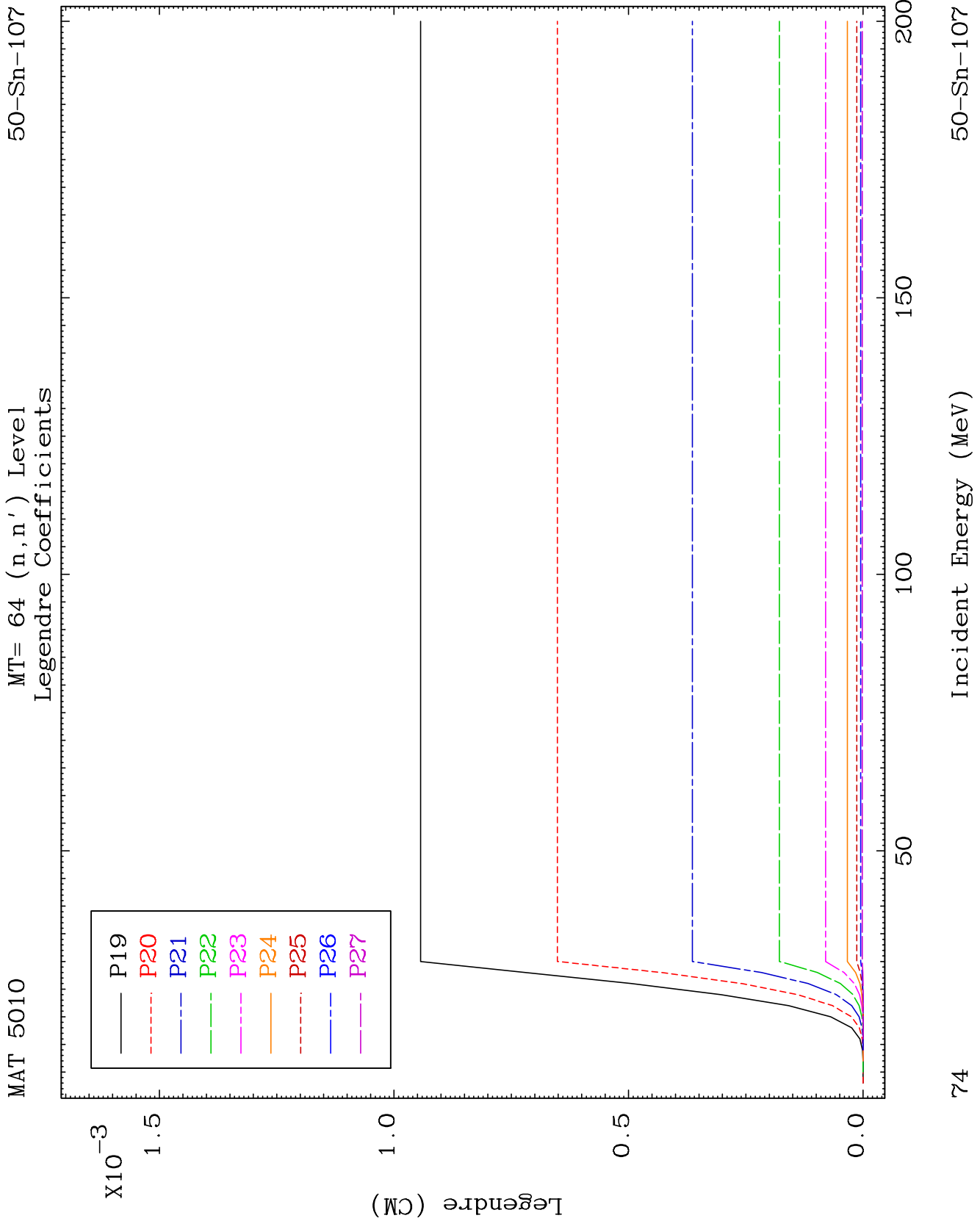


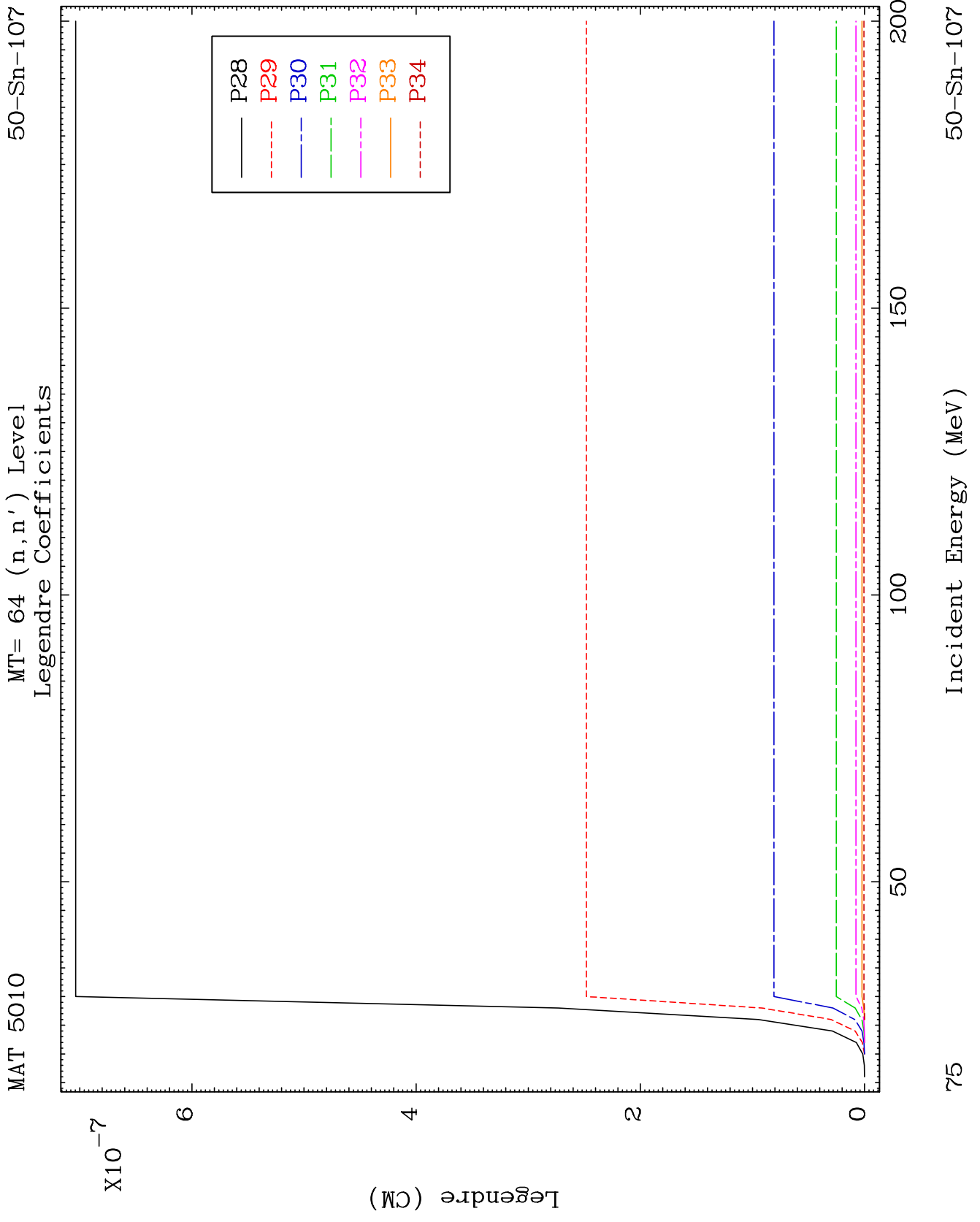


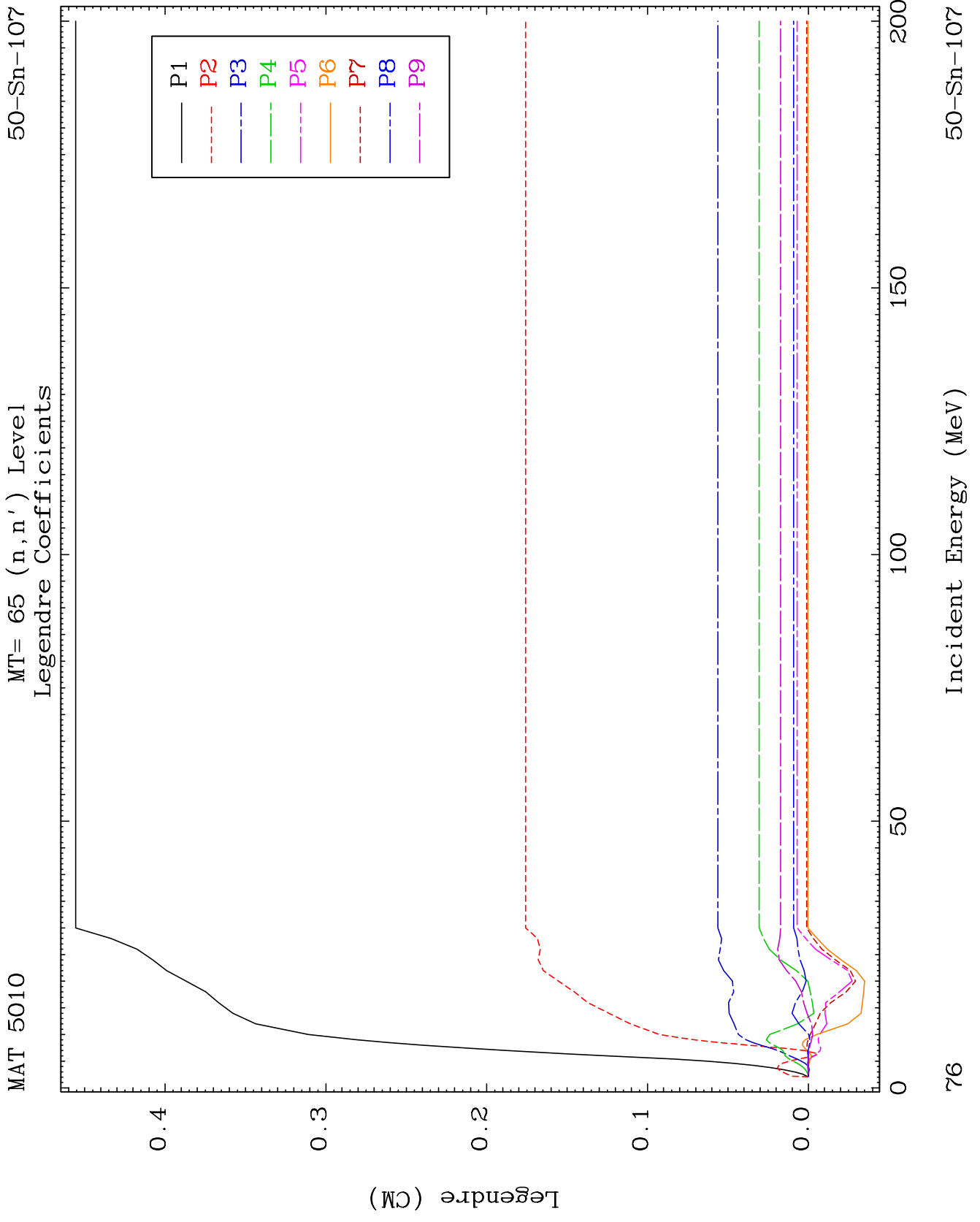


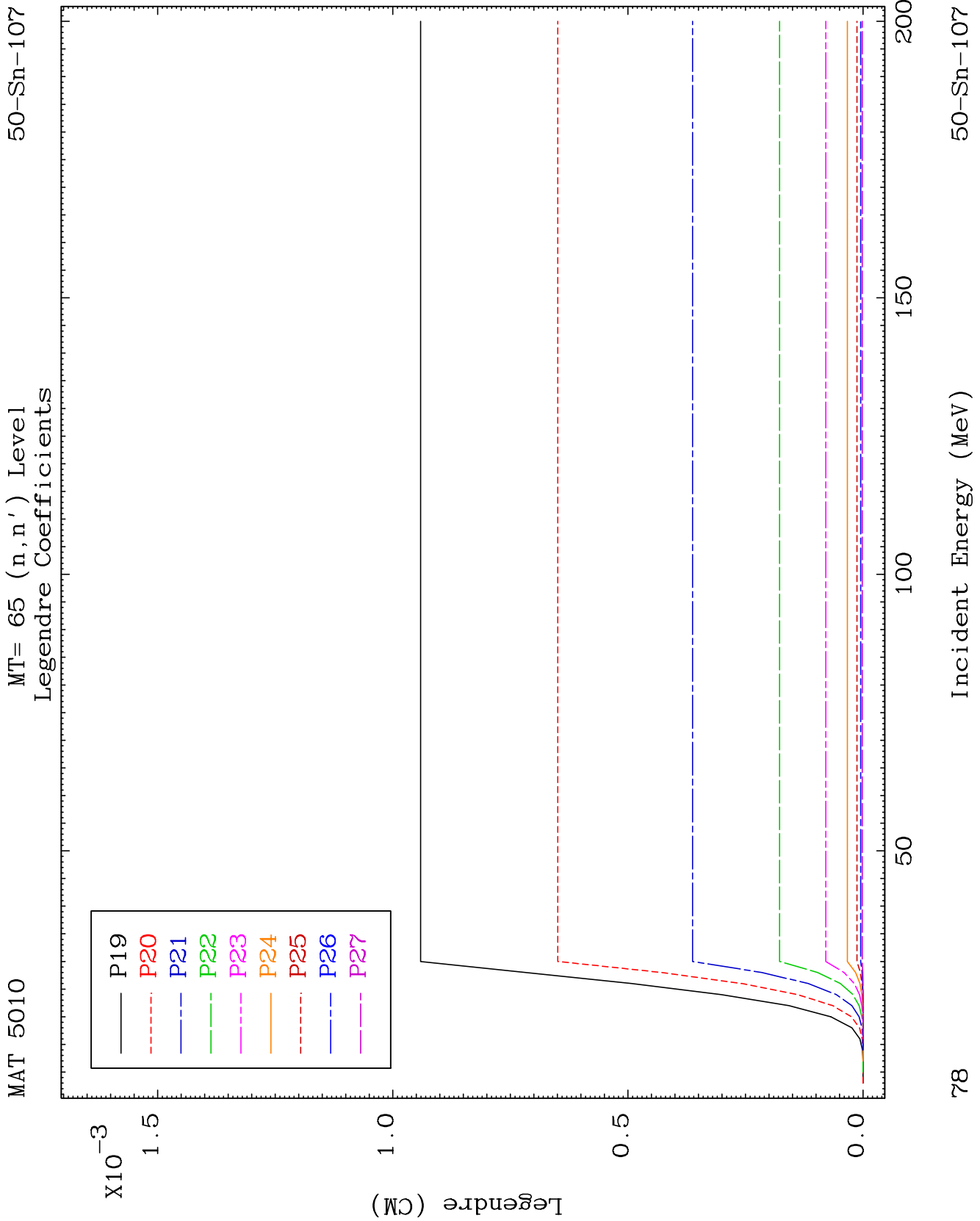


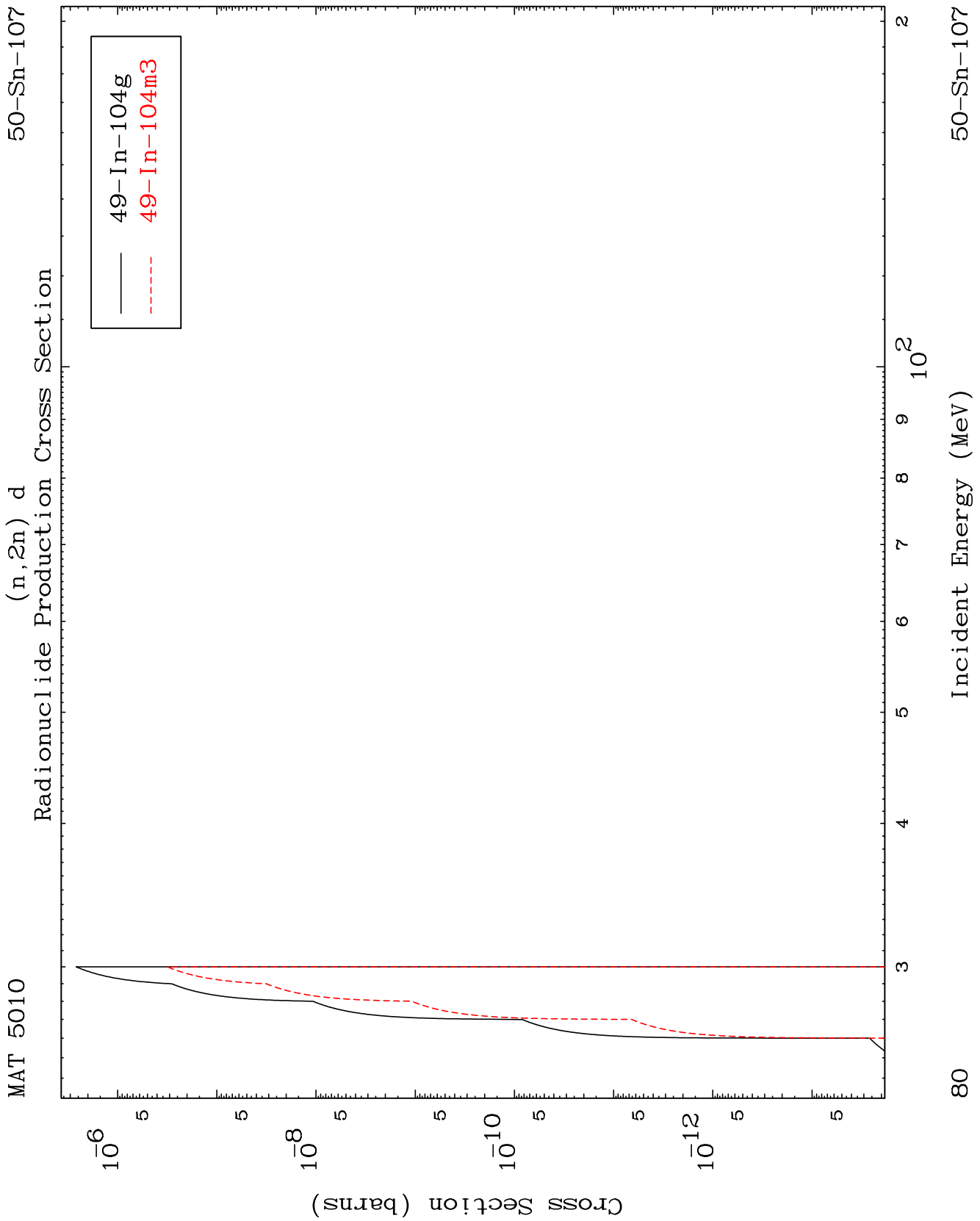










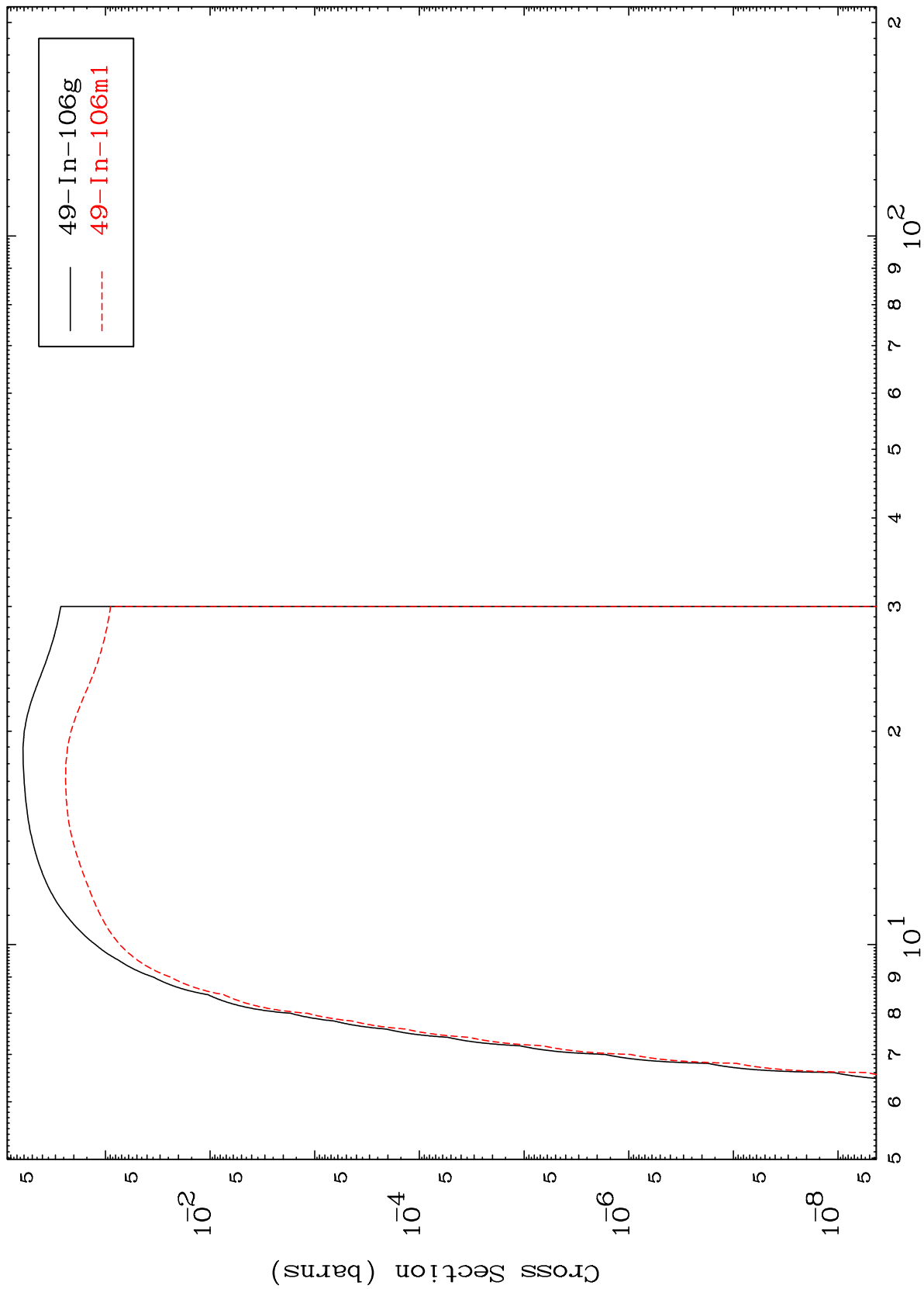


MAT 5010

(n,n') p

50-Sn-107

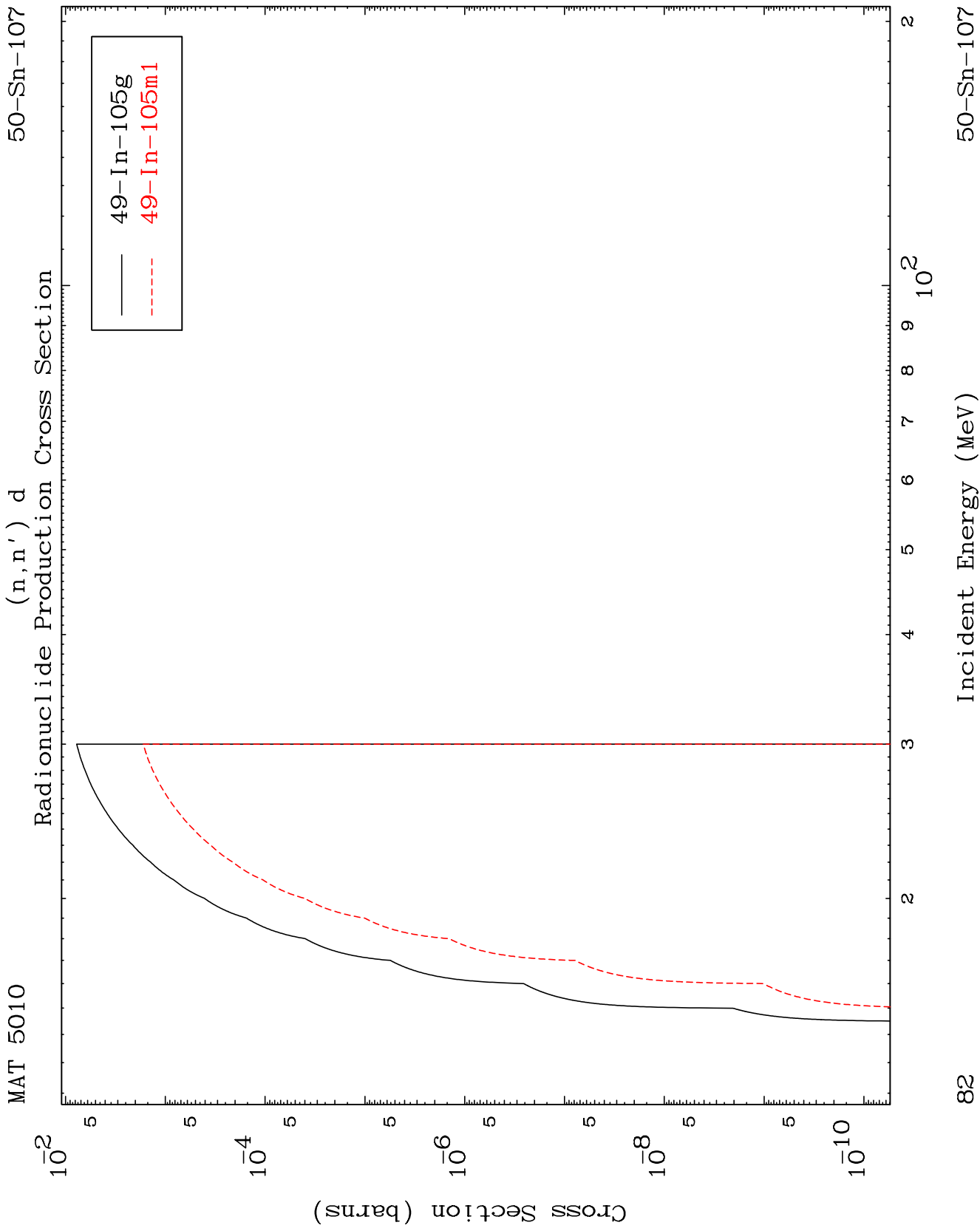
Radionuclide Production Cross Section



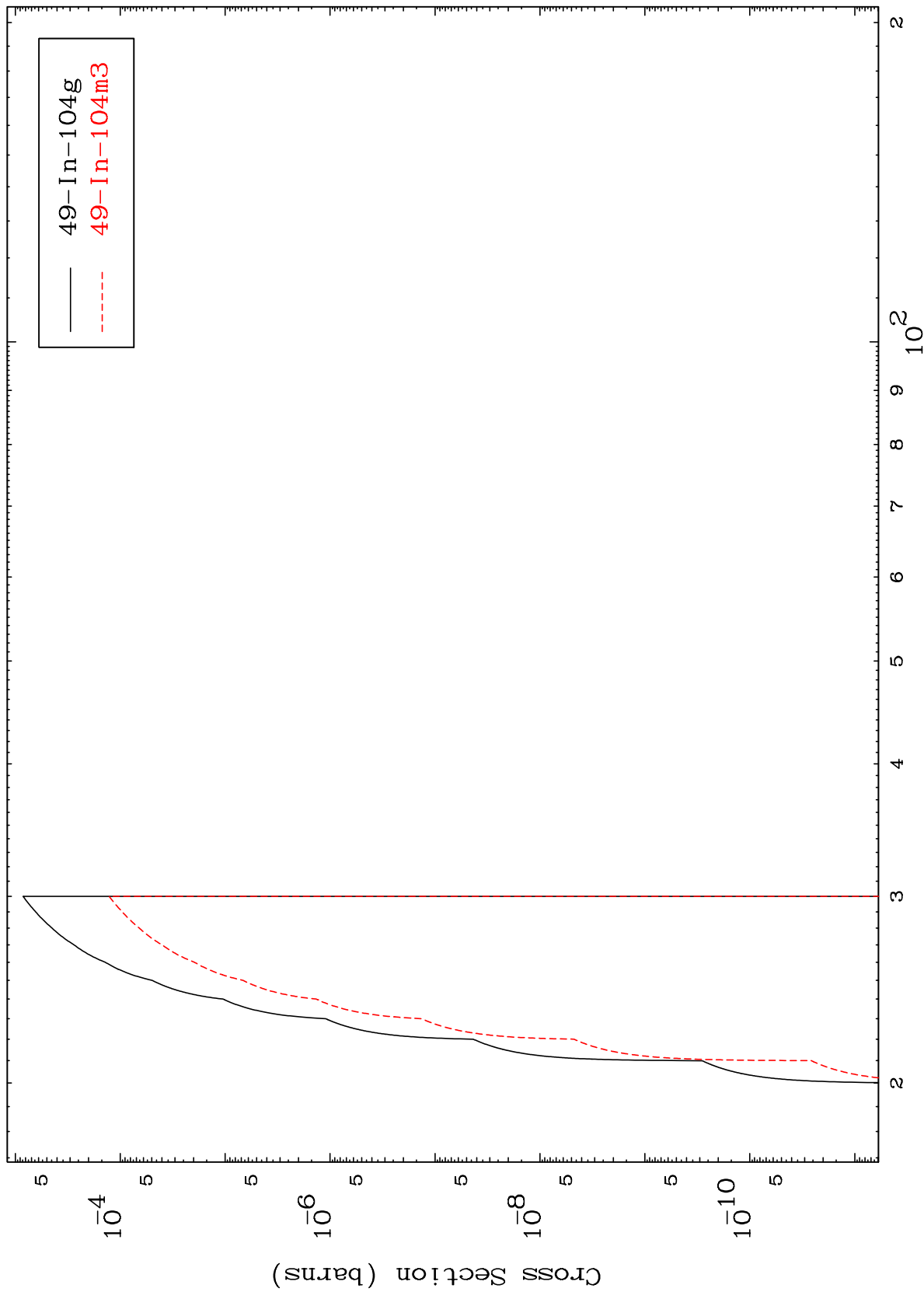
81

Incident Energy (MeV)

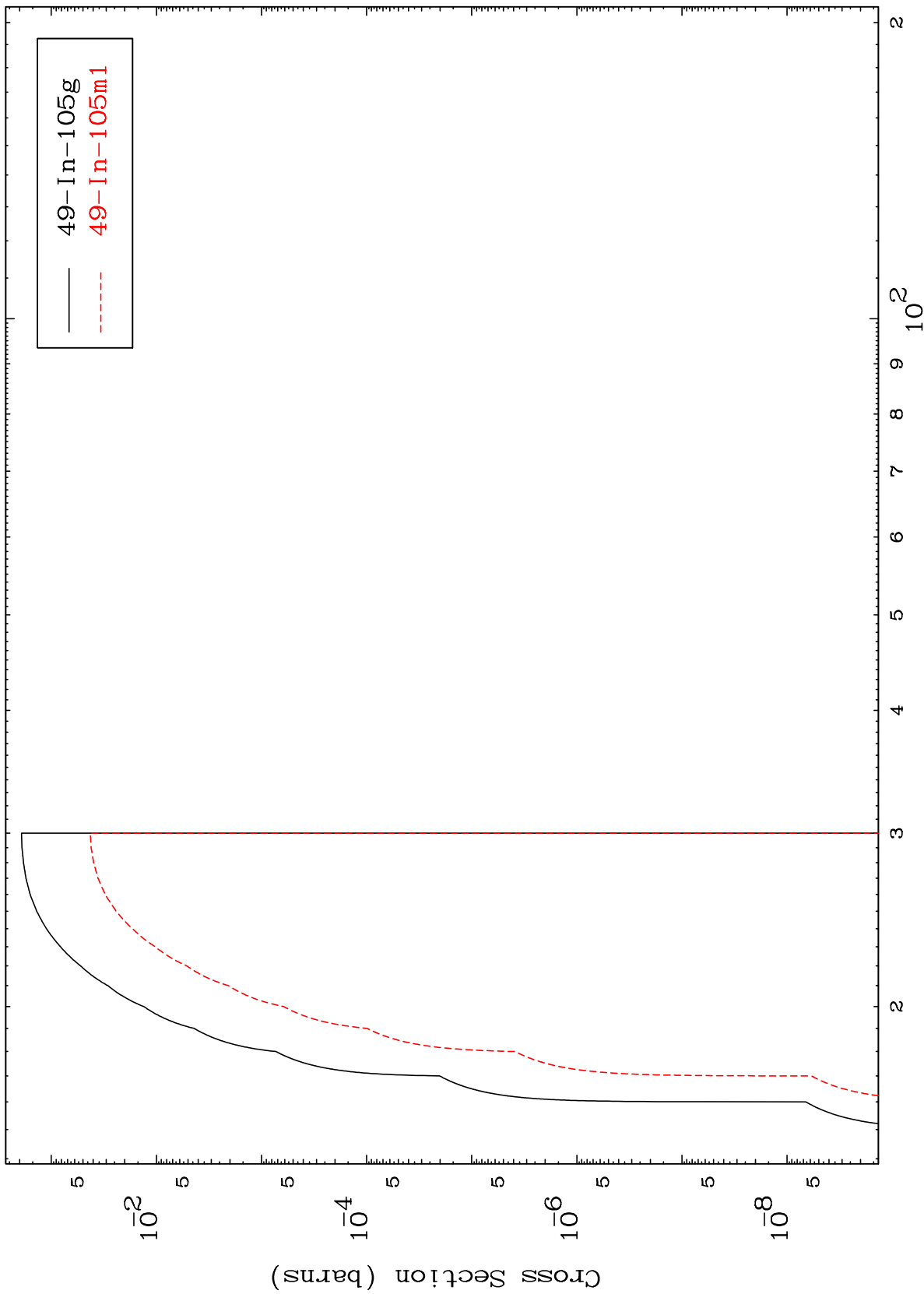
50-Sn-107



Radionuclide Production Cross Section



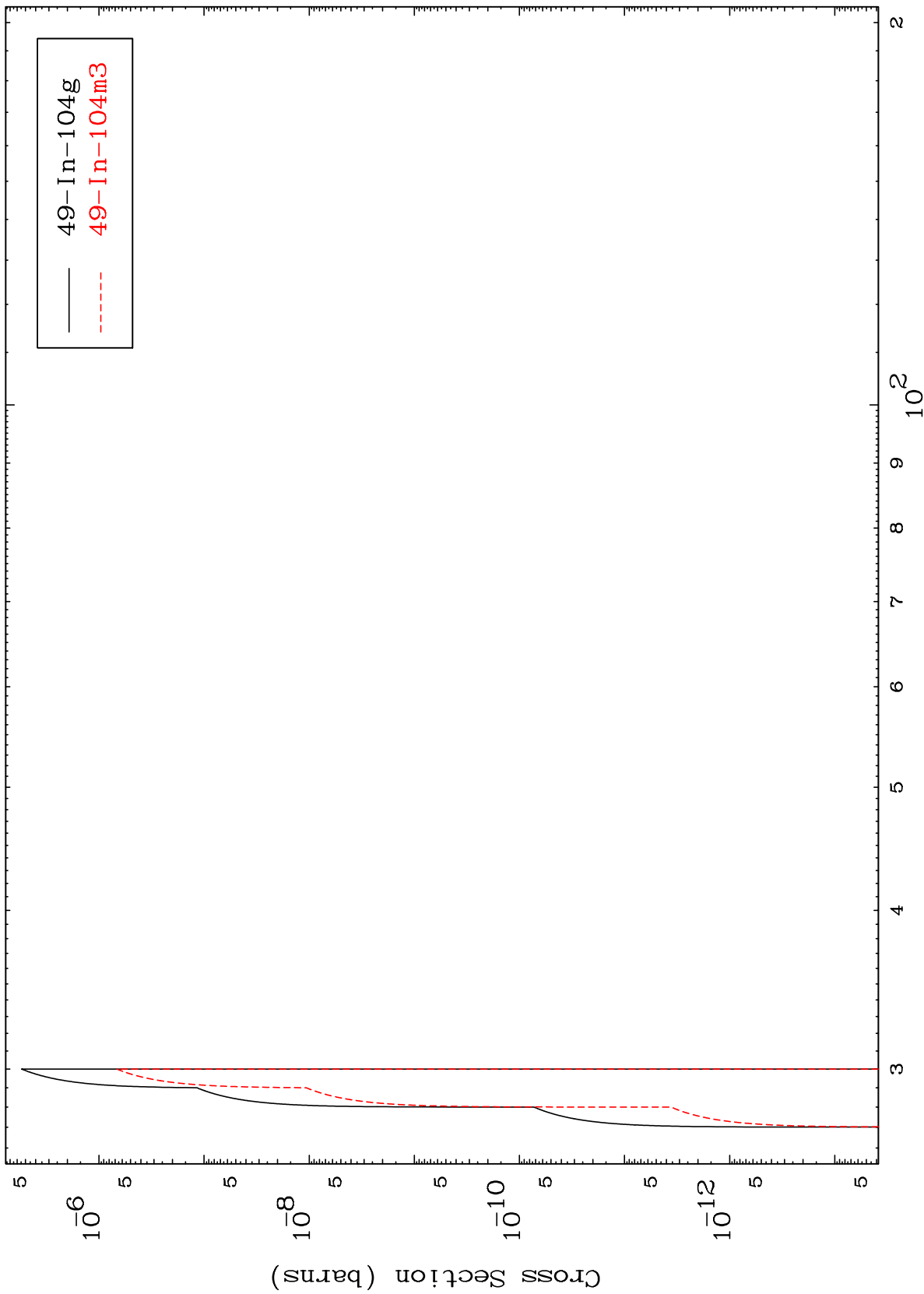
(n,2n) p
Radionuclide Production Cross Section



MAT 5010

50-Sn-107

(n,3n) p
Radionuclide Production Cross Section



85

50-Sn-107

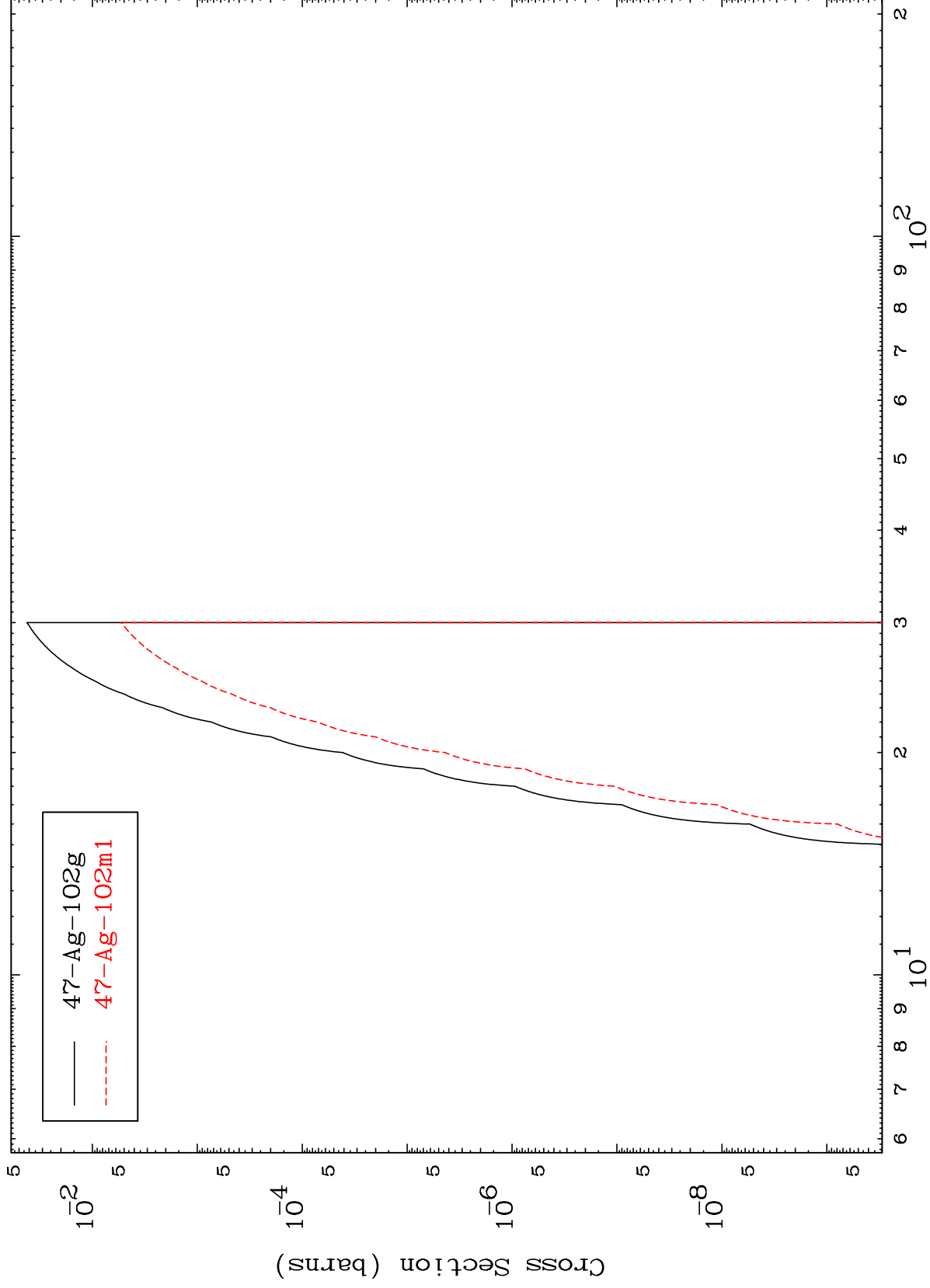
Incident Energy (MeV)

MAT 5010

(n,n') p α

50-Sn-107

Radionuclide Production Cross Section



86

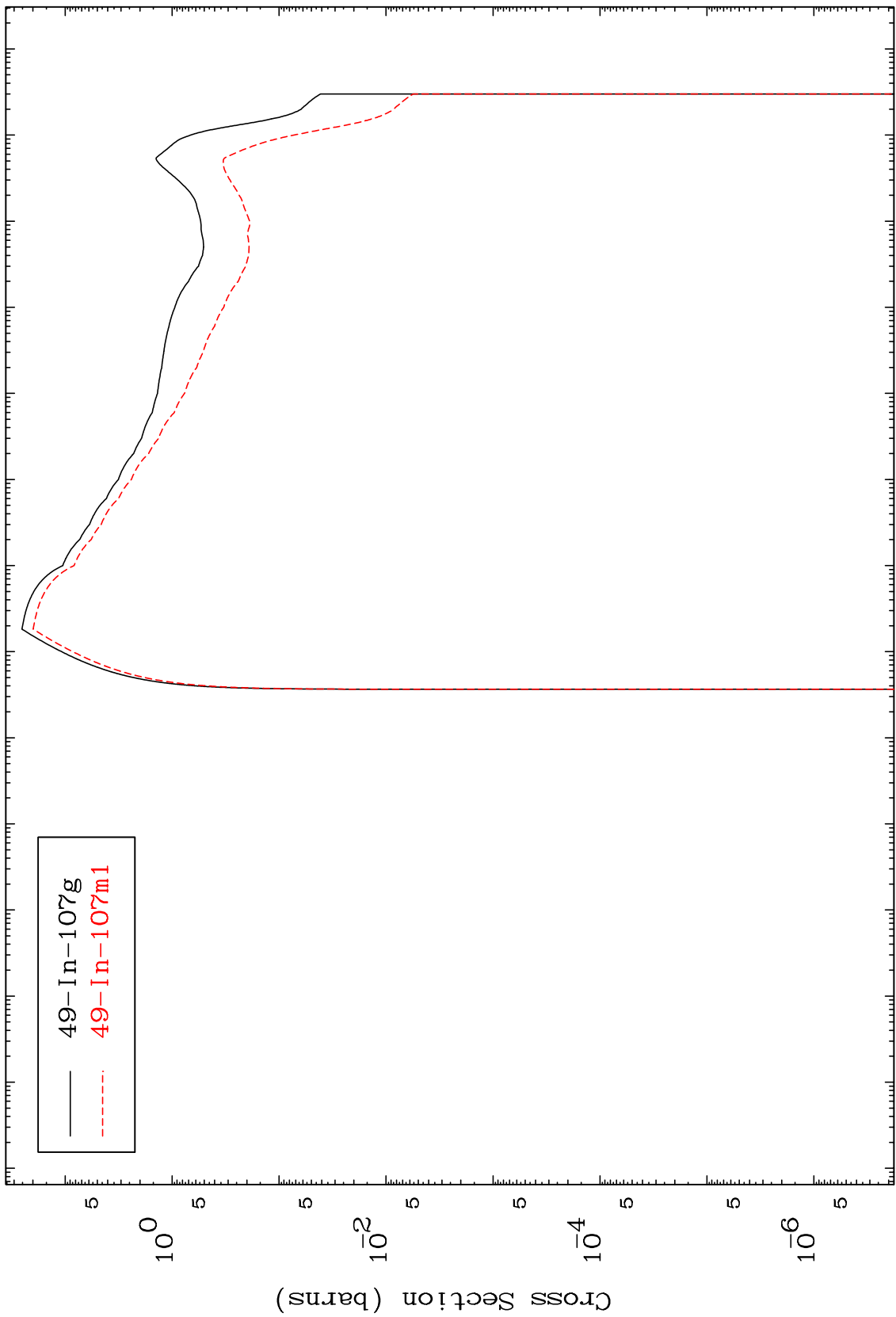
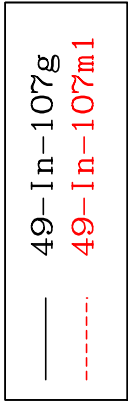
Incident Energy (MeV)

50-Sn-107

MAT 5010

50-Sn-107

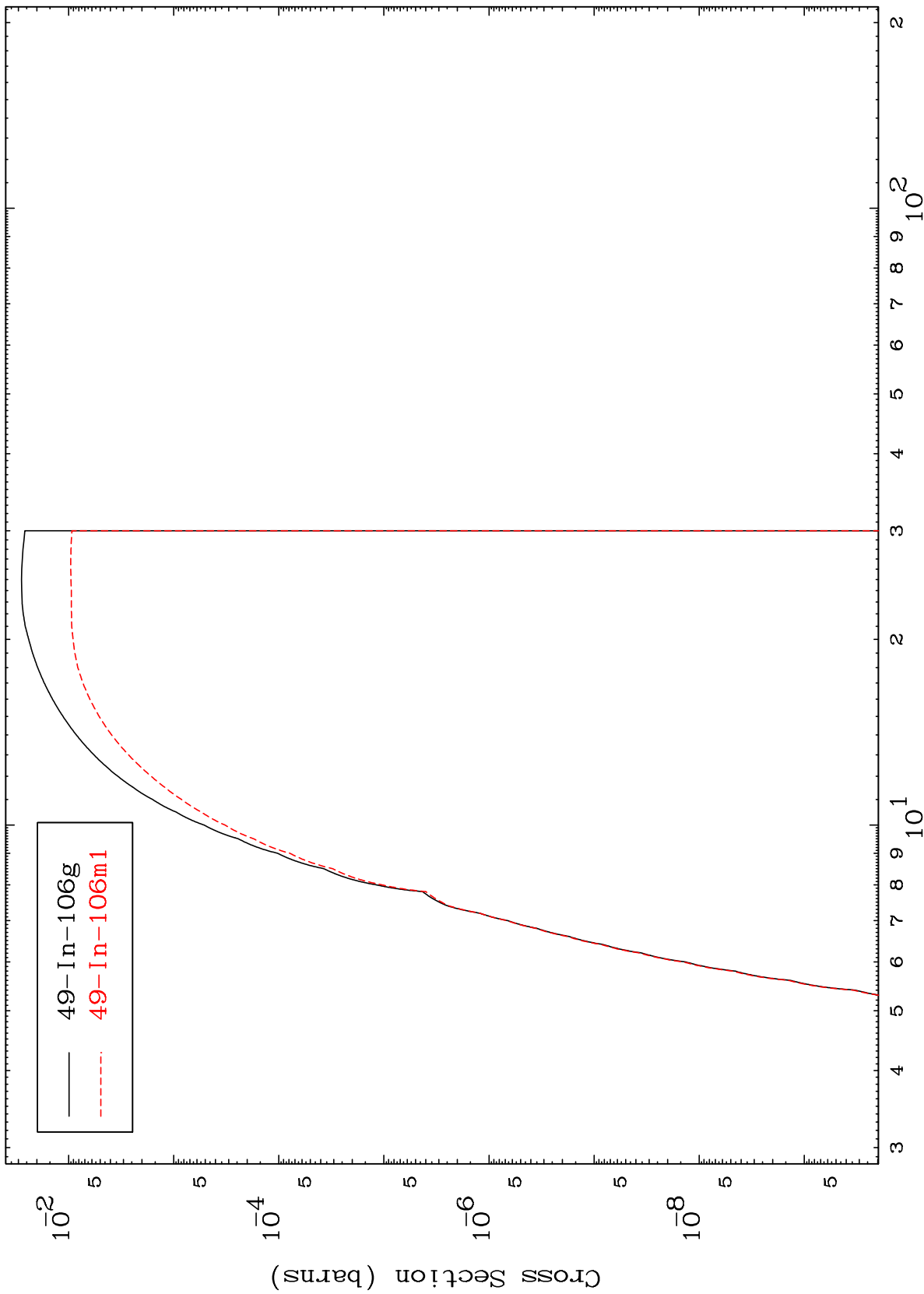
(n,p)
Radionuclide Production Cross Section



MAT 5010

50-Sn-107

(n,d)
Radionuclide Production Cross Section



88

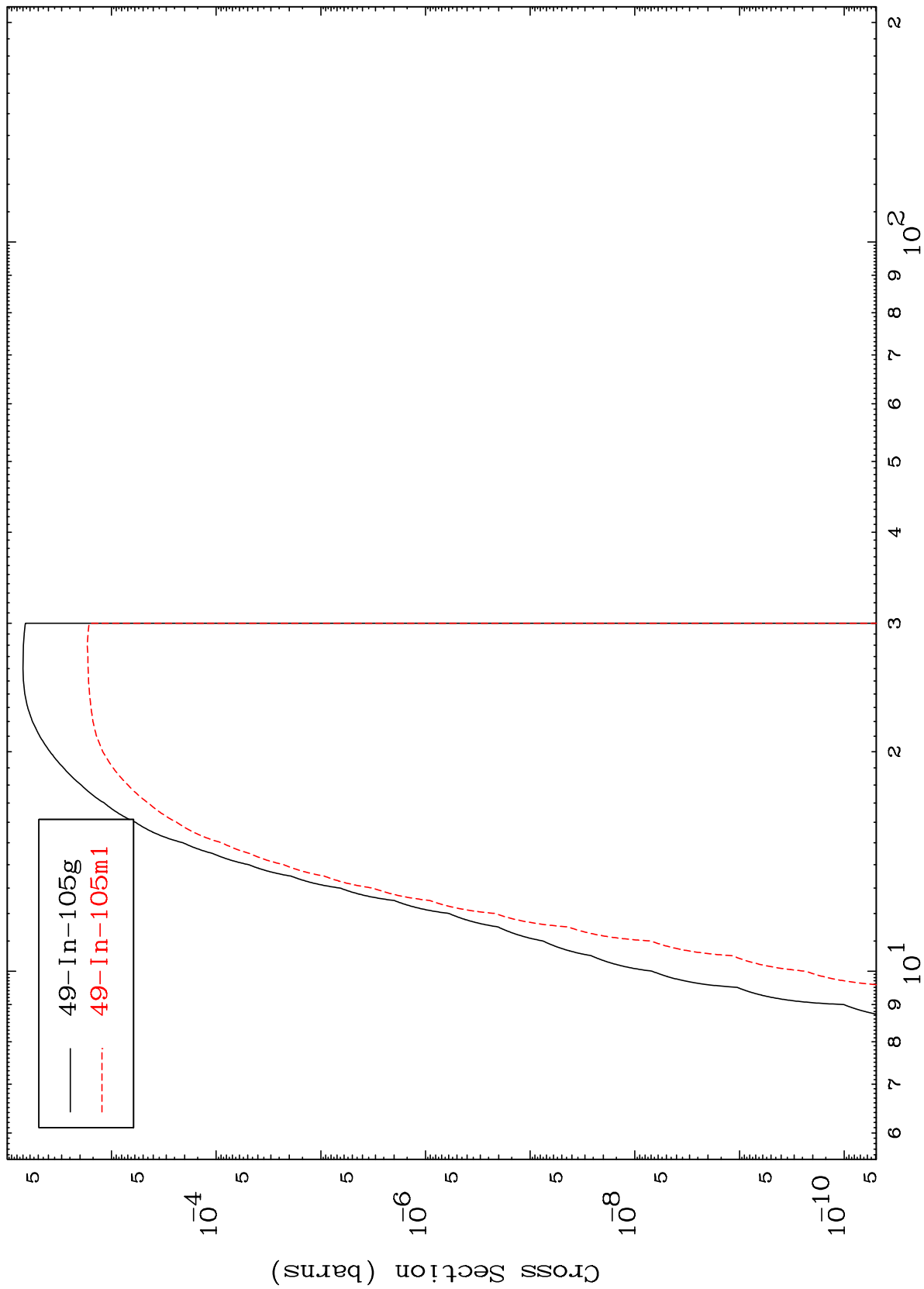
50-Sn-107

Incident Energy (MeV)

MAT 5010

50-Sn-107

(n, t)
Radionuclide Production Cross Section



— 49-In-105g
- - - 49-In-105m1

89

Incident Energy (MeV)

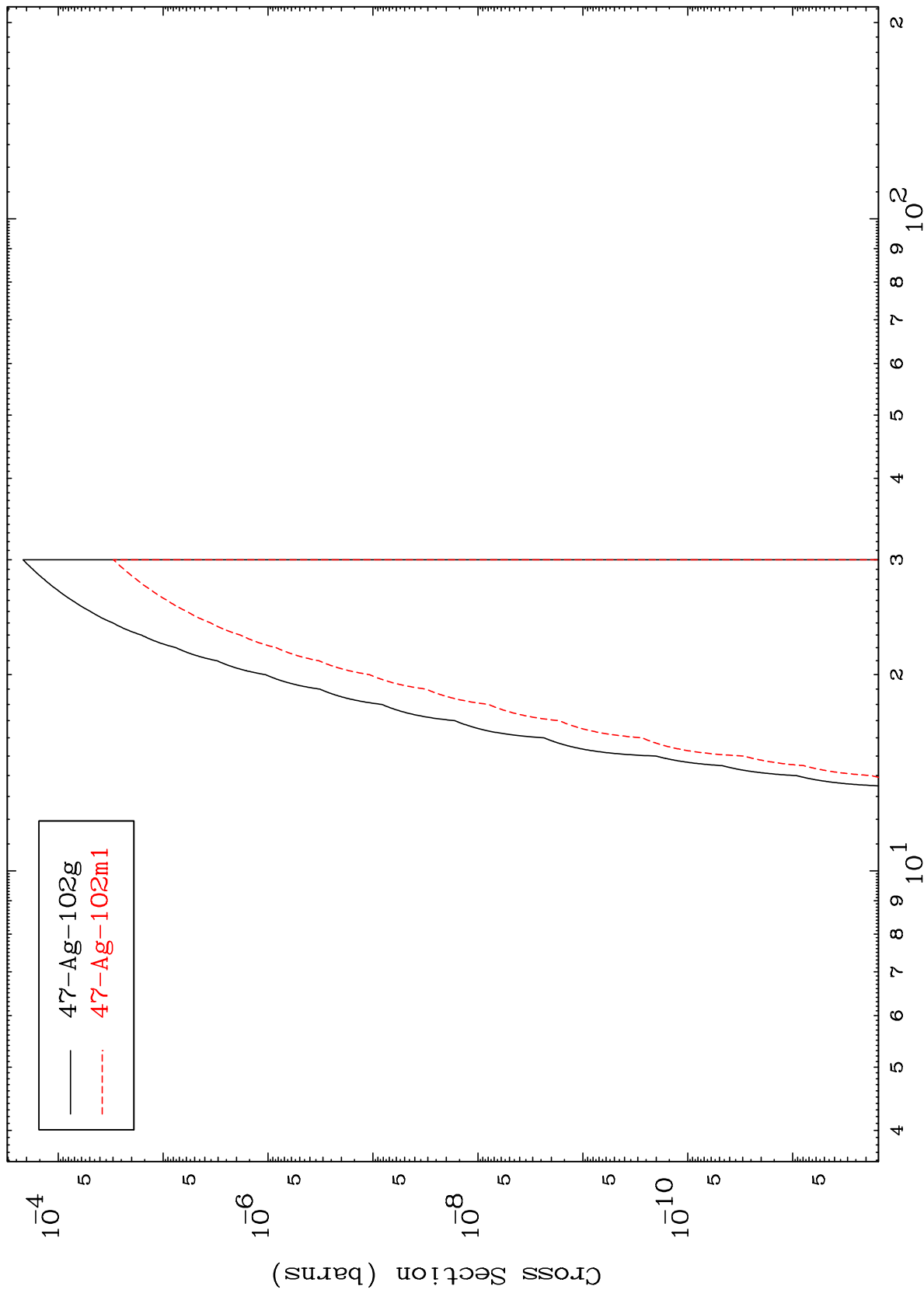
50-Sn-107

MAT 5010

50-Sn-107

(n,d) α

Radionuclide Production Cross Section



91

Incident Energy (MeV)

50-Sn-107