

Program EVALPLOT
(Version 2021-1)

by

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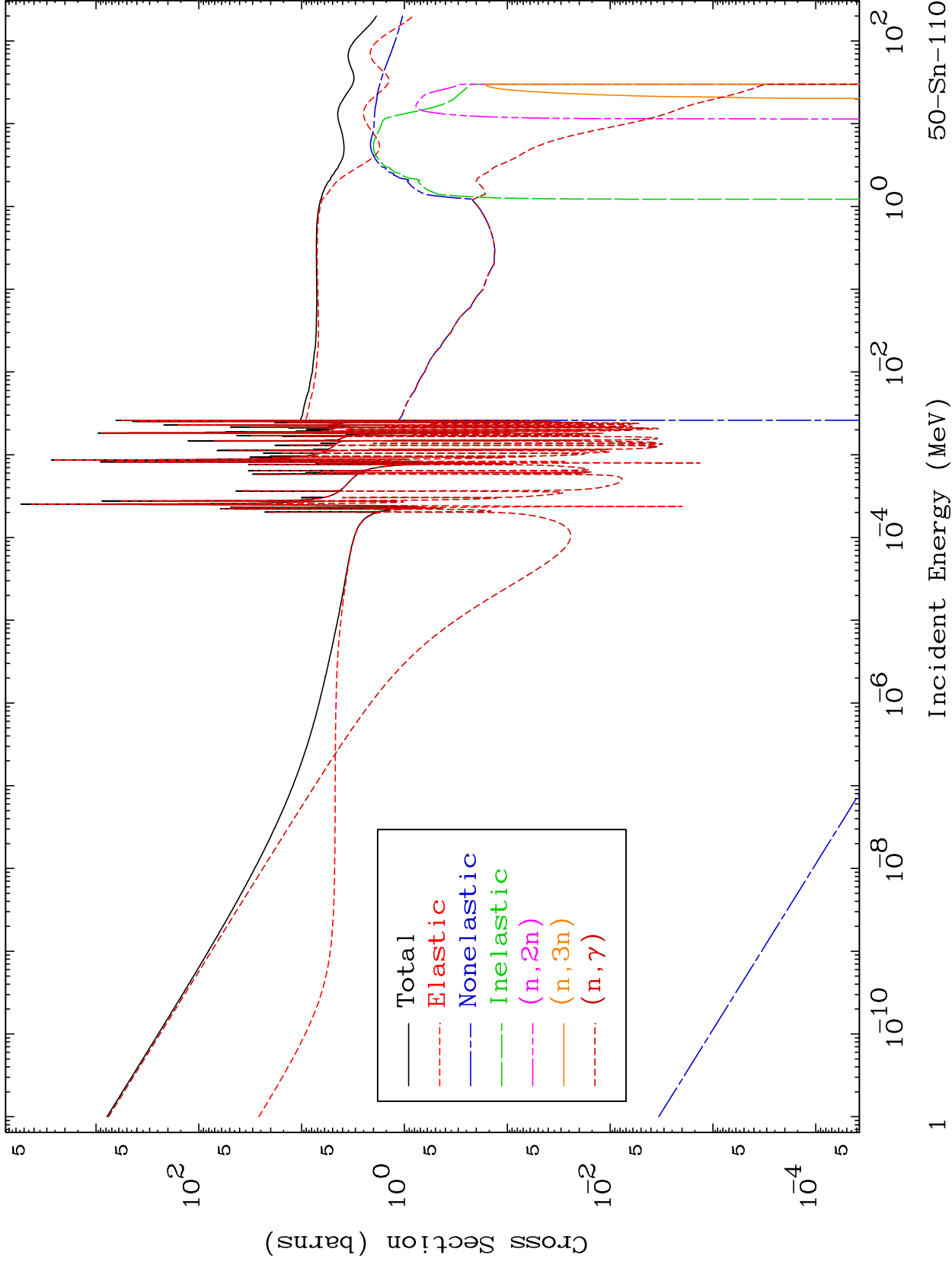
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Press Mouse Button to Start

MAT 5019

Neutron Major
293 Kelvin Cross Sections

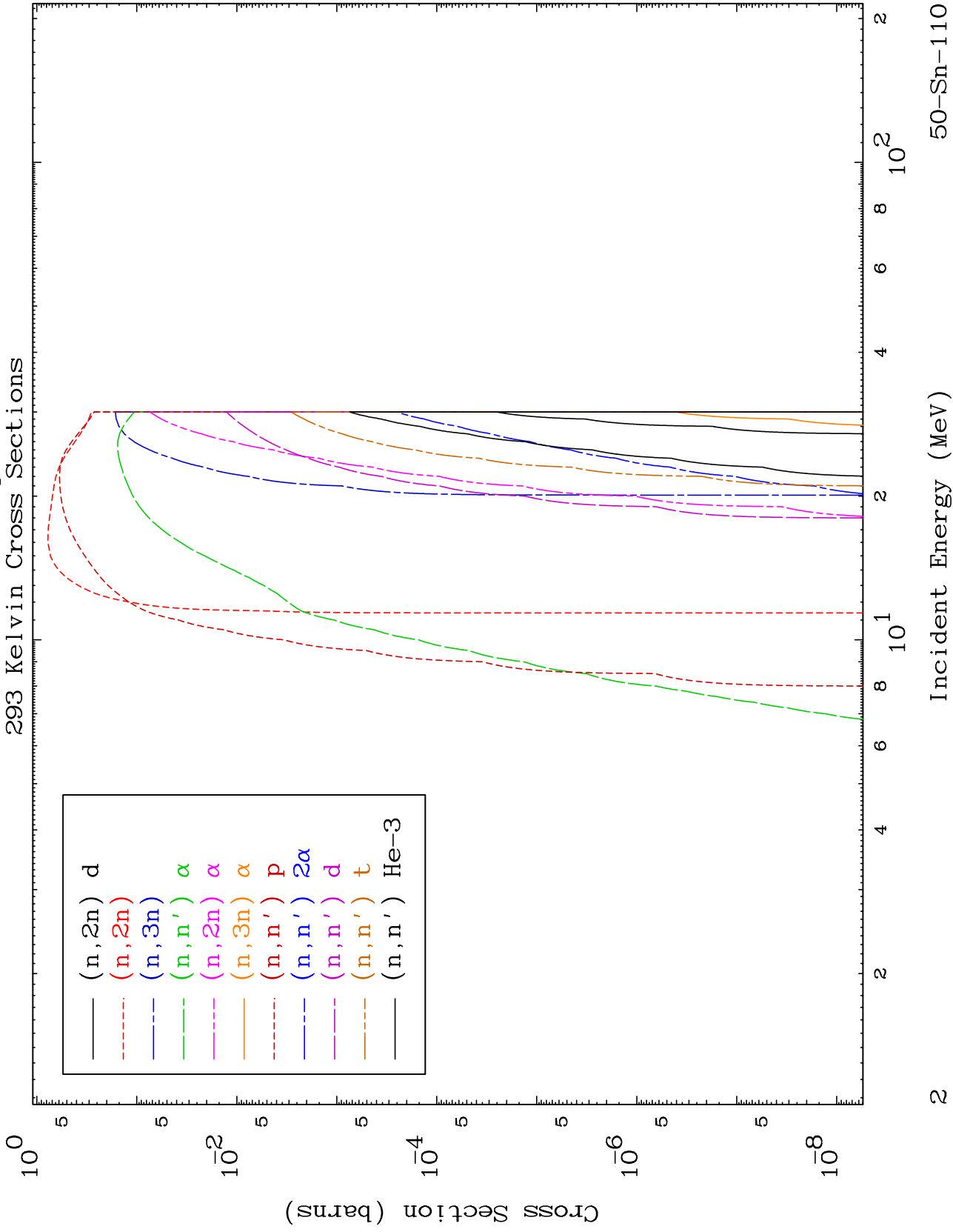
50-Sn-110



MAT 5019

Neutron Absorption
293 Kelvin Cross Sections

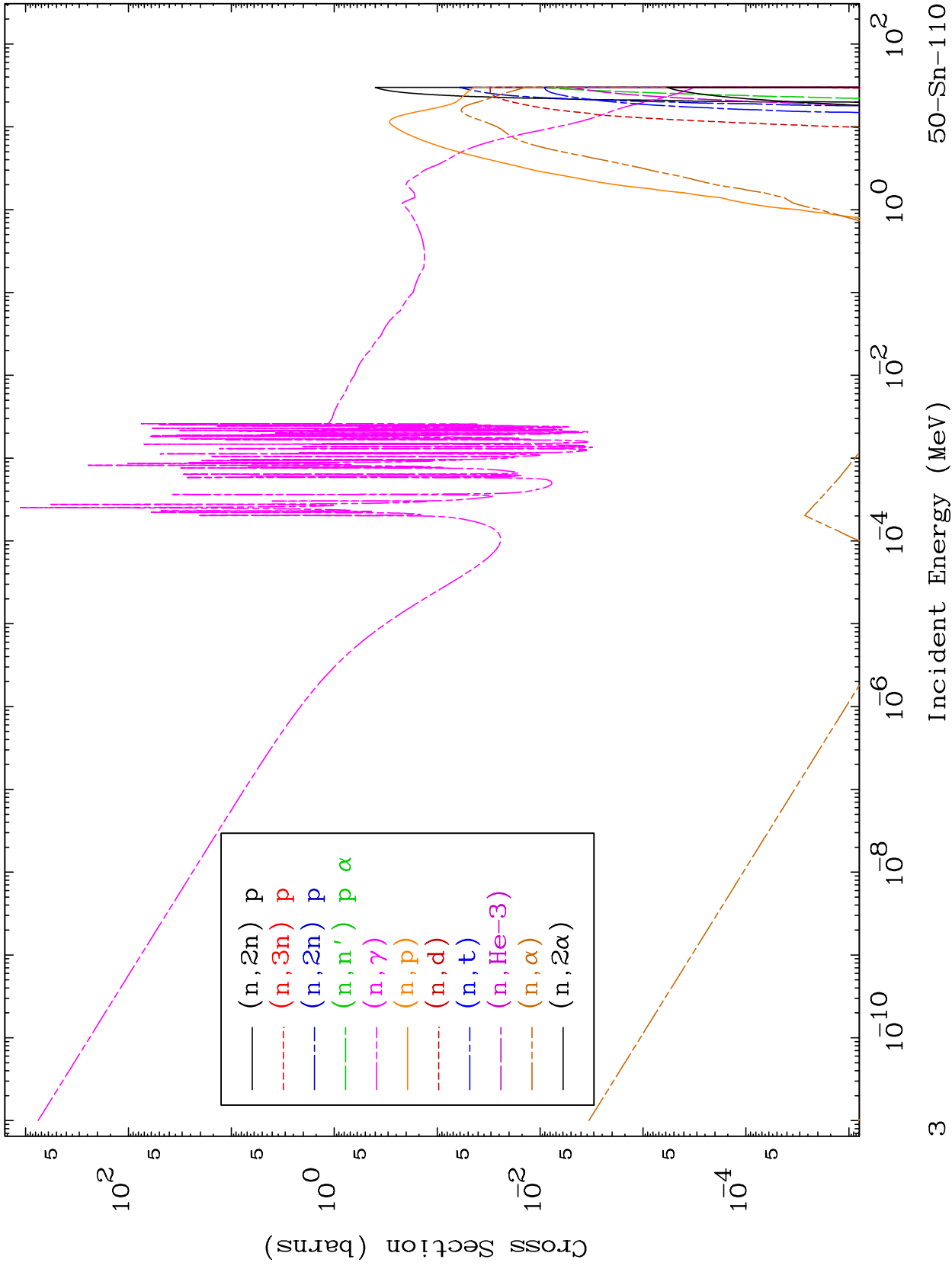
50-Sn-110

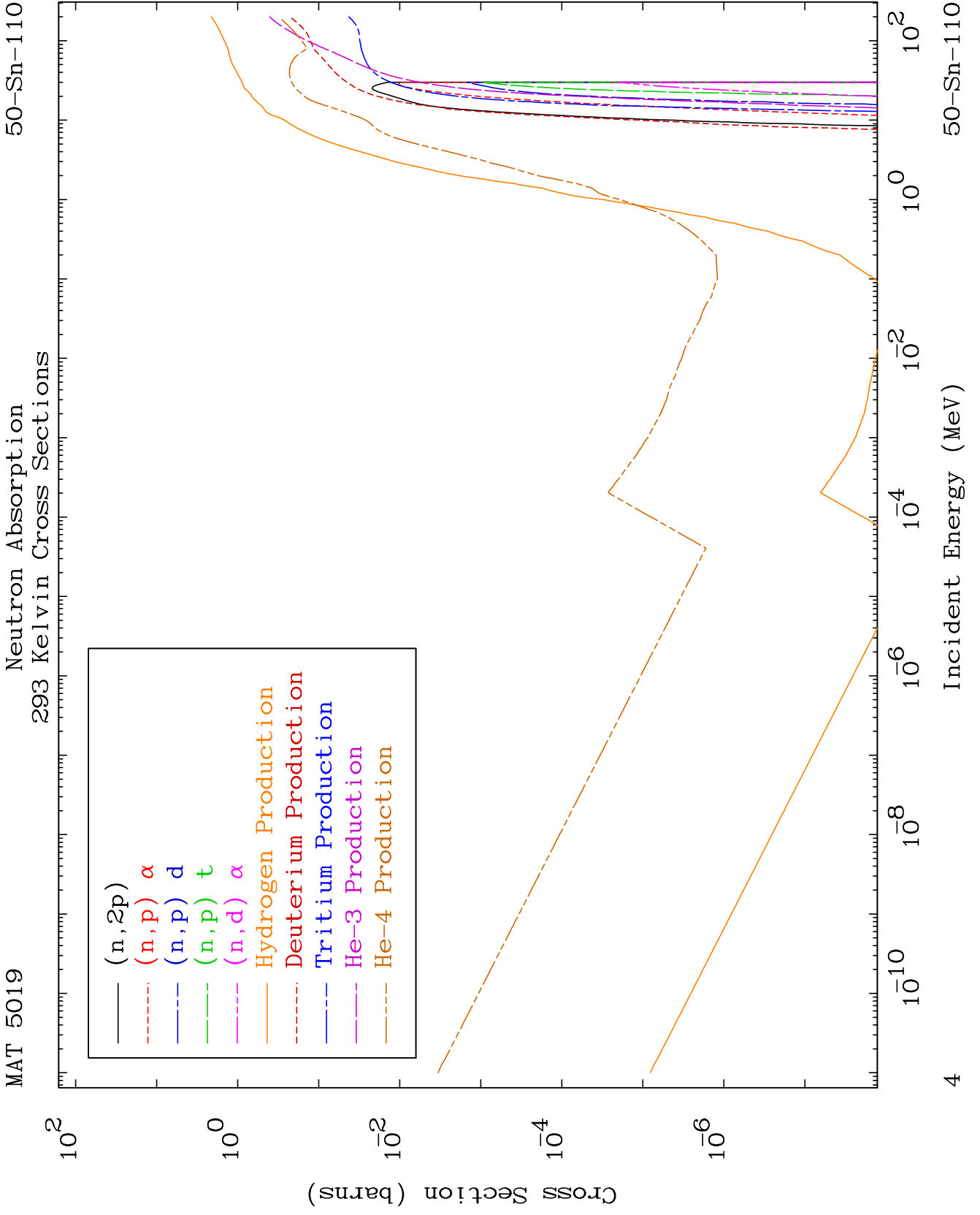


MAT 5019

Neutron Absorption
293 Kelvin Cross Sections

50-Sn-110

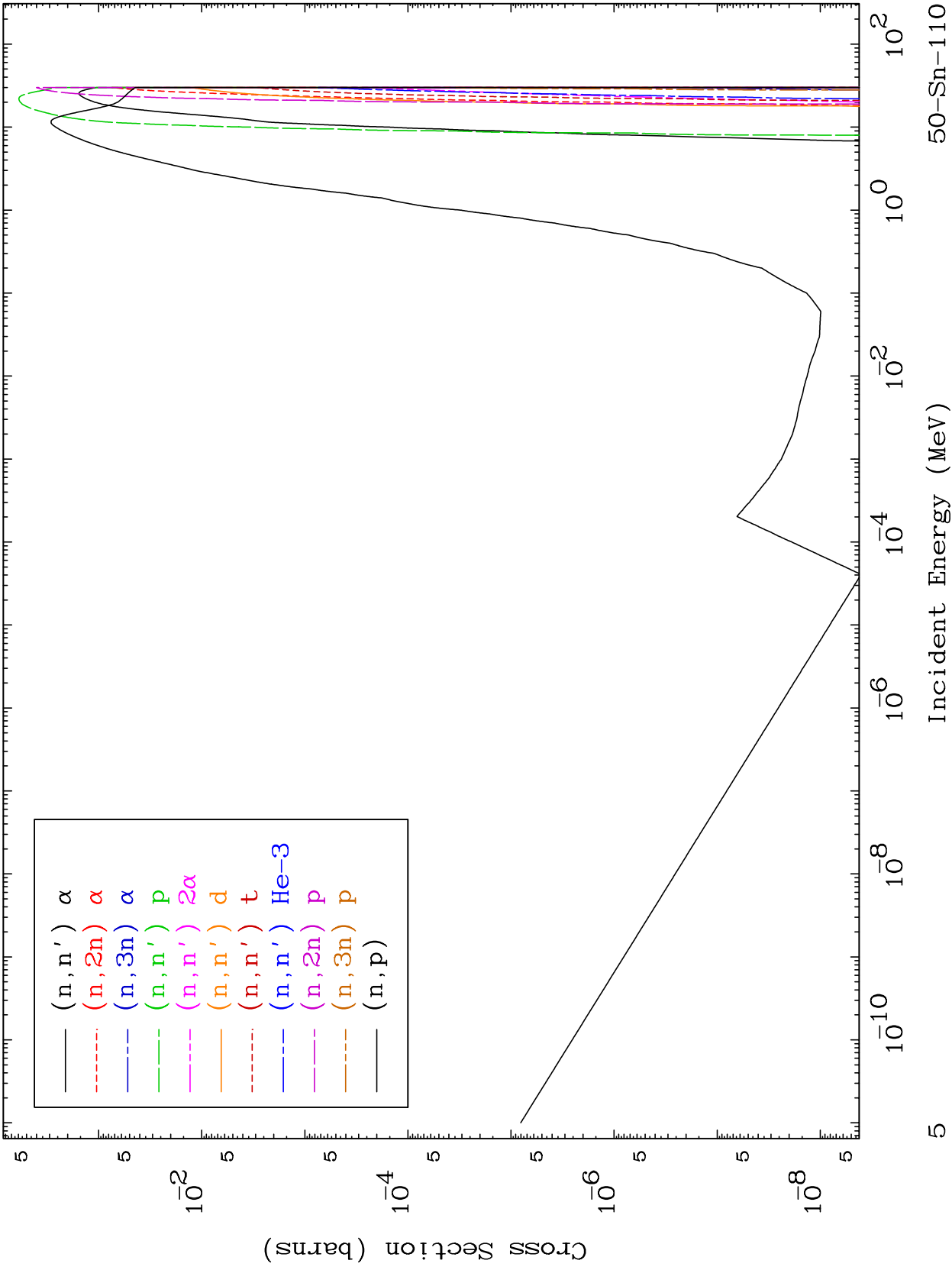




MAT 5019

Charged Particle
293 Kelvin Cross Sections

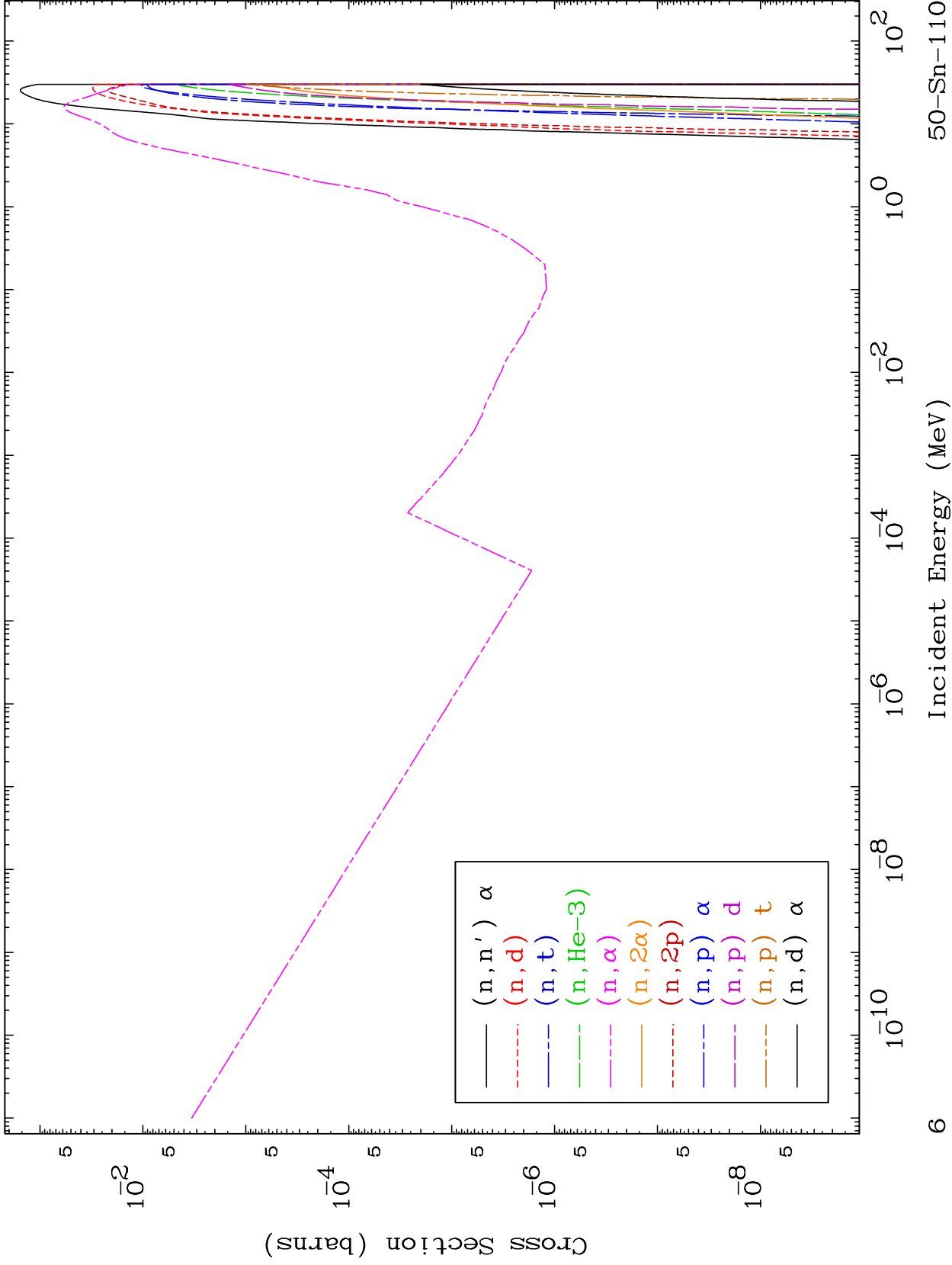
50-Sn-110



MAT 5019

Charged Particle
293 Kelvin Cross Sections

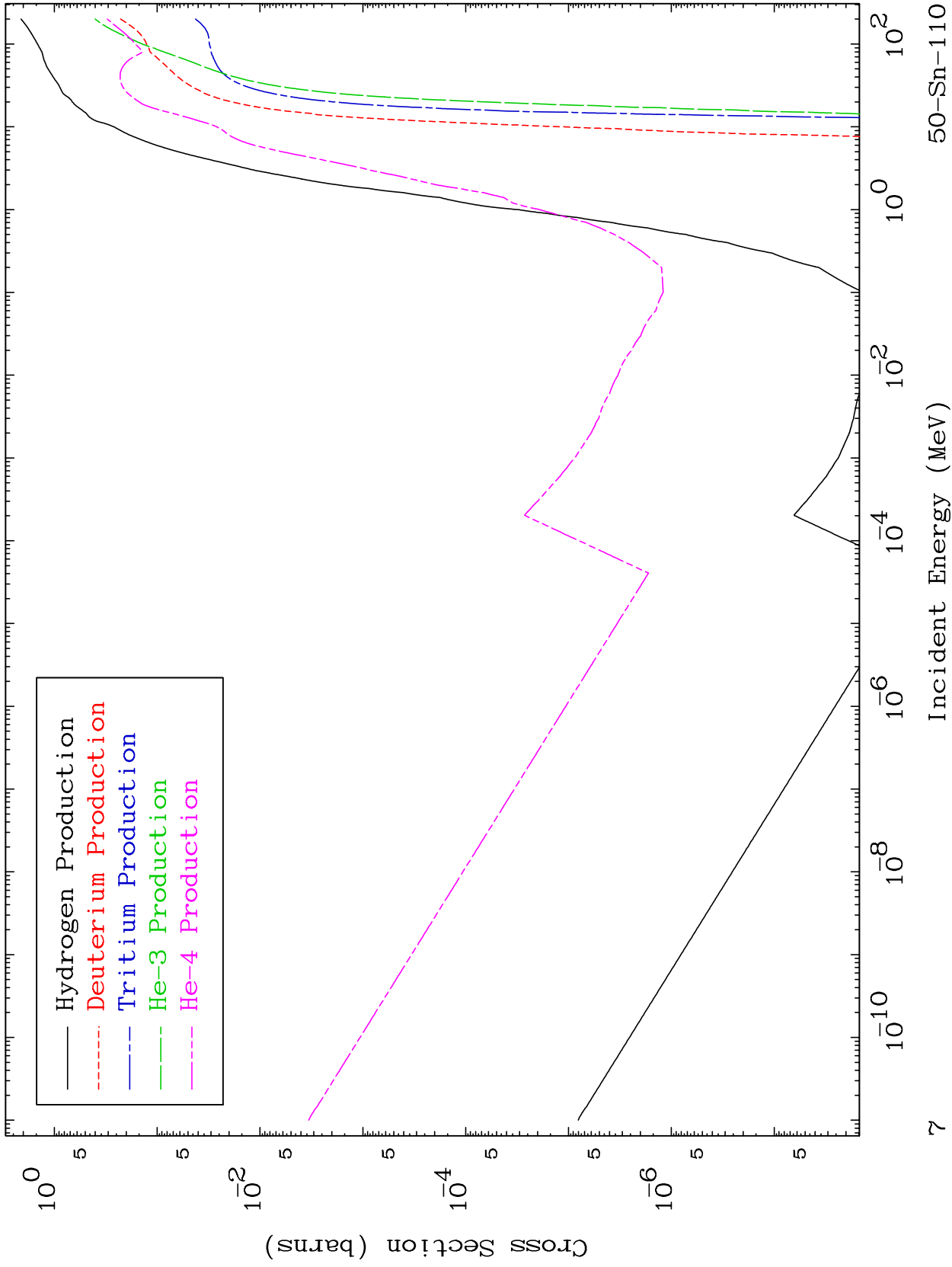
50-Sn-110



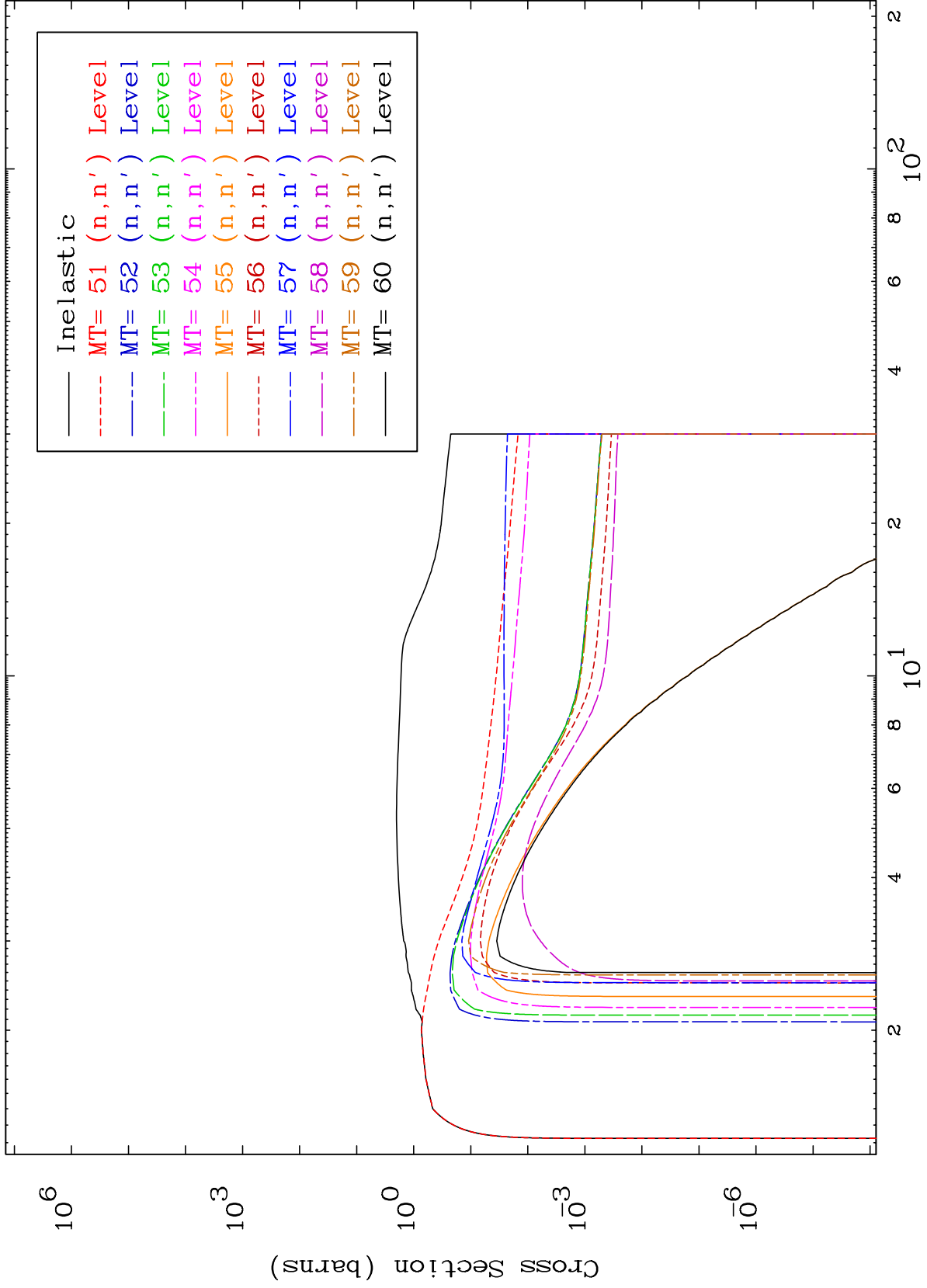
MAT 5019

Particle Production
293 Kelvin Cross Sections

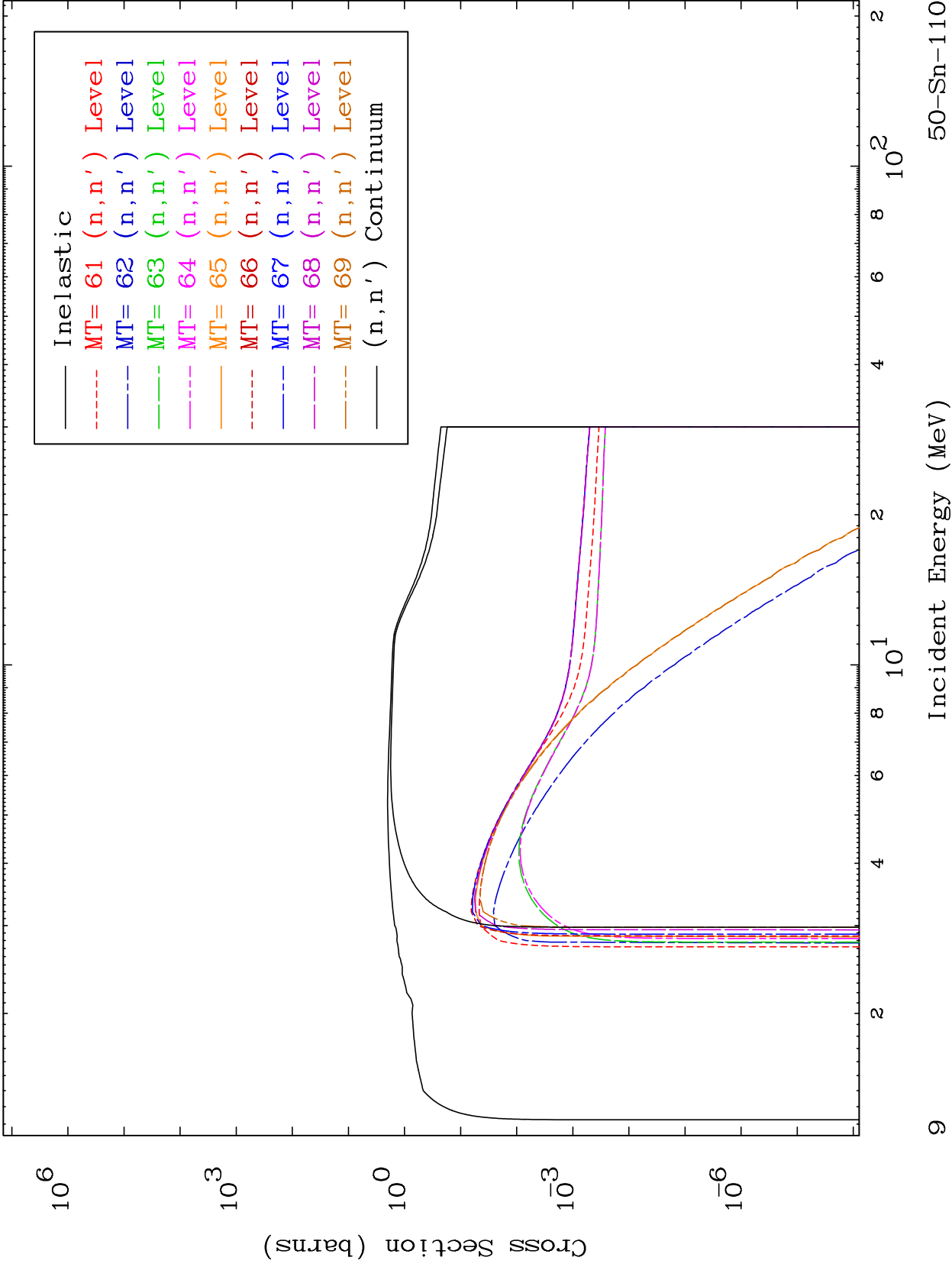
50-Sn-110



293 Kelvin Cross Sections



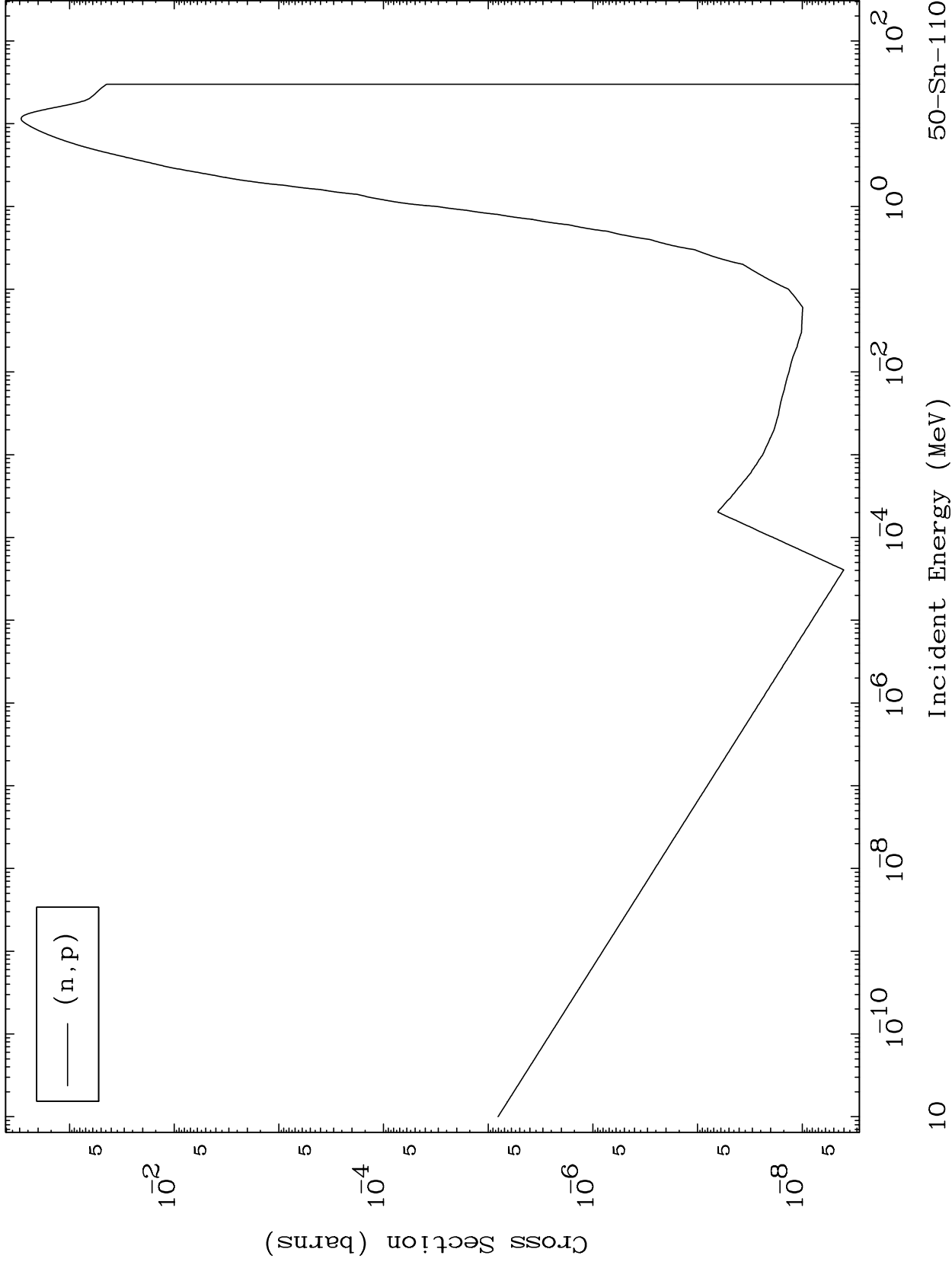
293 Kelvin Cross Sections



MAT 5019

(n,p) Levels
293 Kelvin Cross Sections

50-Sn-110



(n,p)

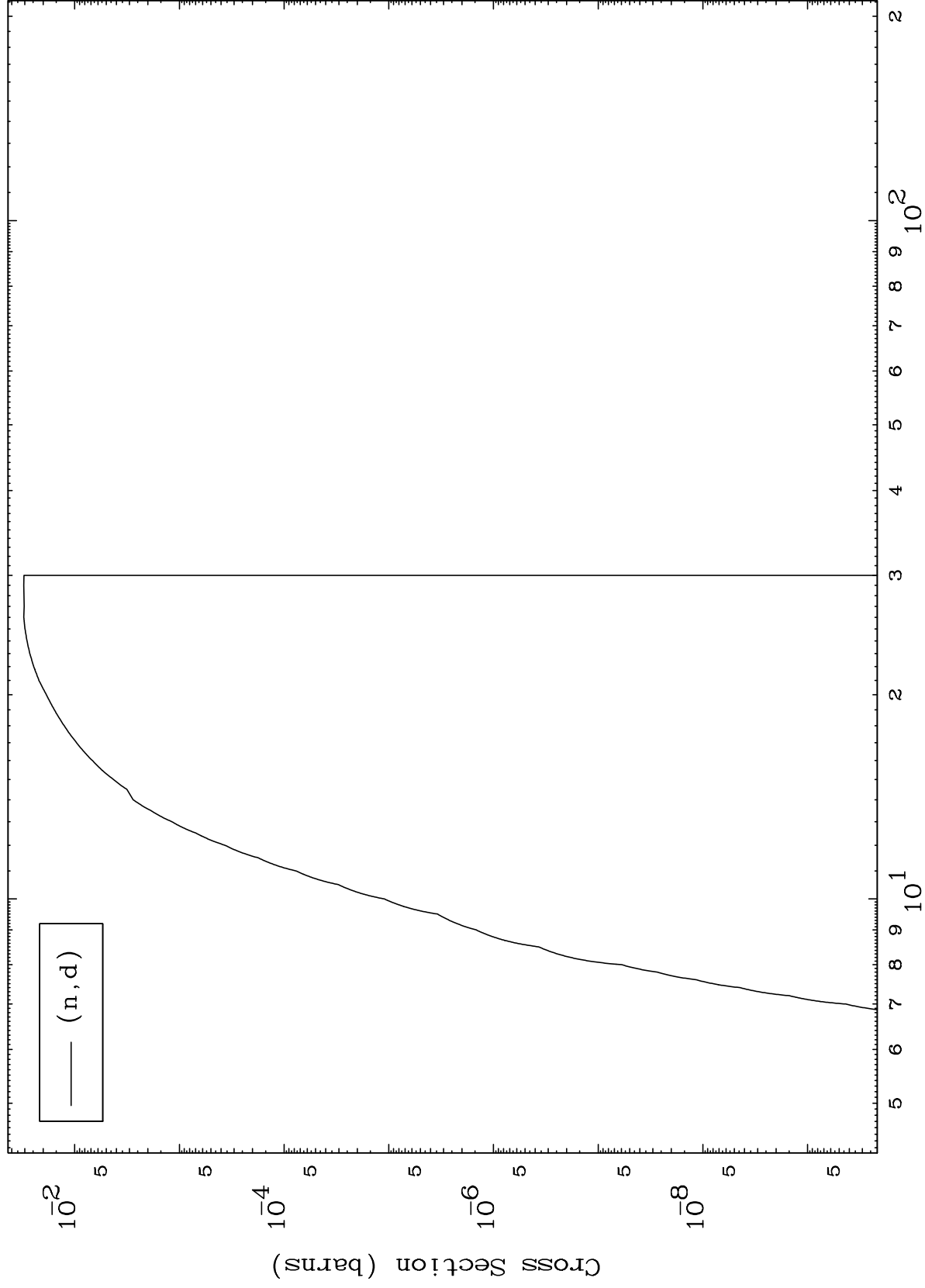
10

50-Sn-110

MAT 5019

(n,d) Levels
293 Kelvin Cross Sections

50-Sn-110



11

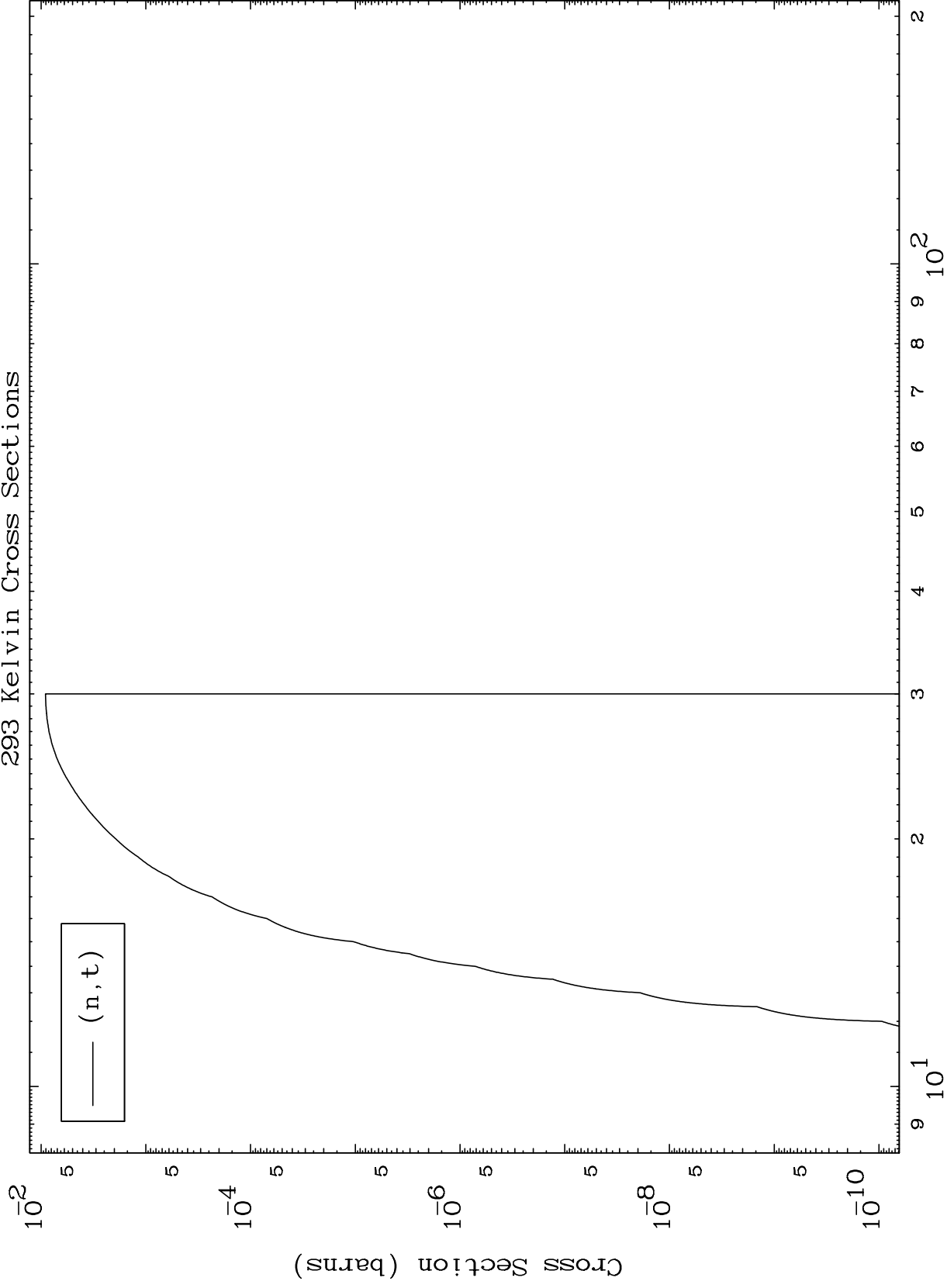
Incident Energy (MeV)

50-Sn-110

MAT 5019

(n,t) Levels
293 Kelvin Cross Sections

50-Sn-110



12

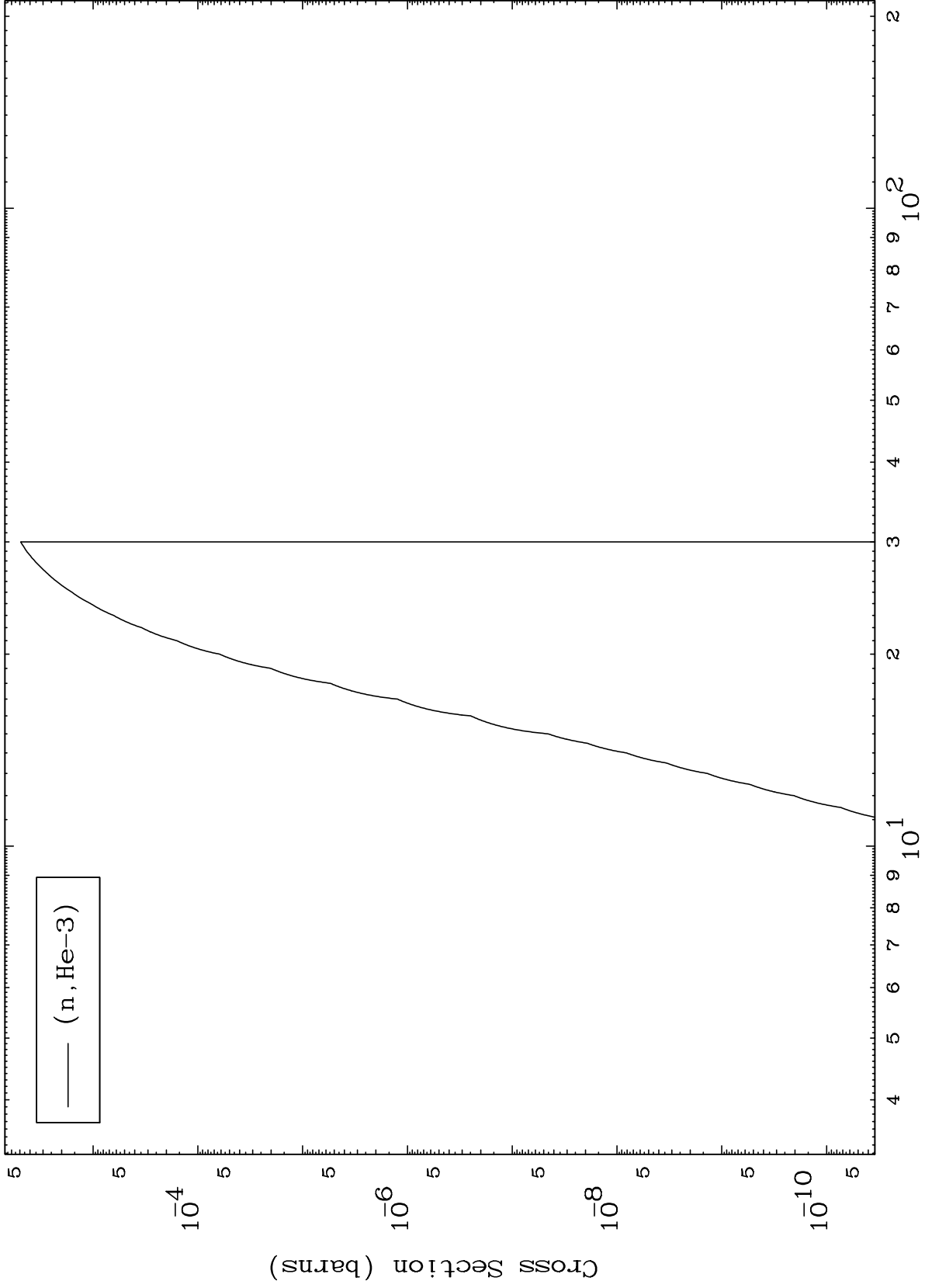
Incident Energy (MeV)

50-Sn-110

MAT 5019

(n,He3) Levels
293 Kelvin Cross Sections

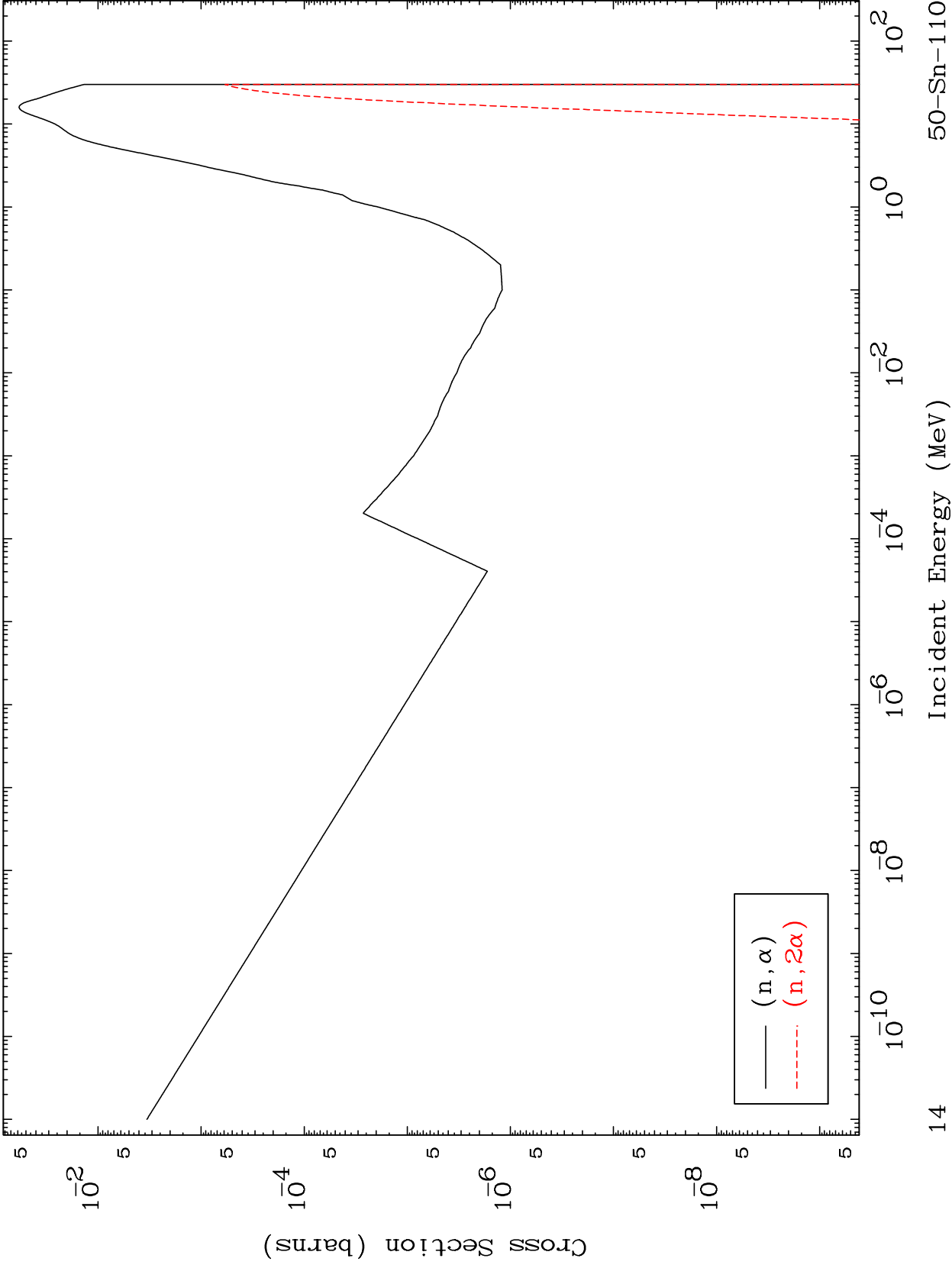
50-Sn-110



MAT 5019

(n,α) Levels
293 Kelvin Cross Sections

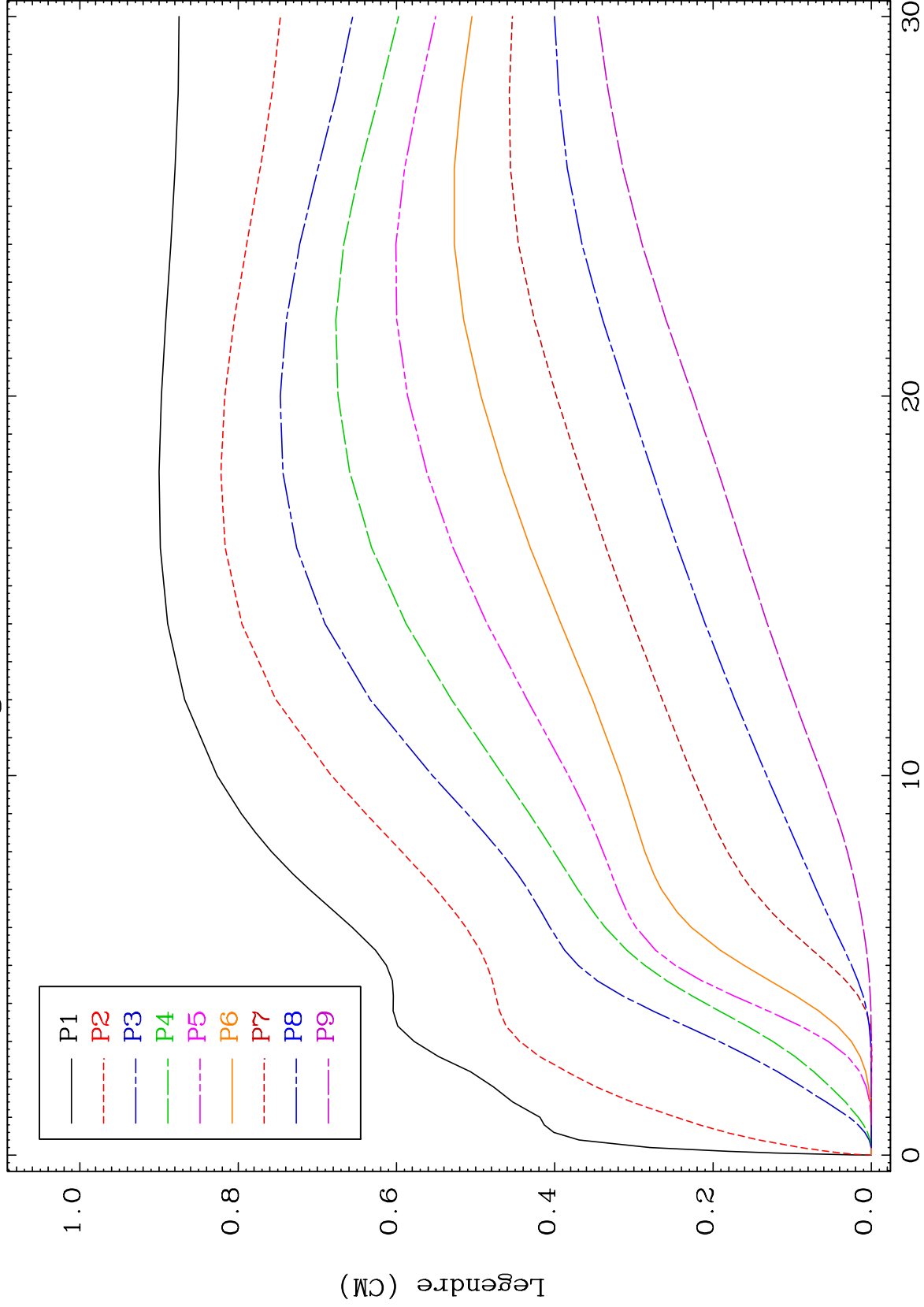
50-Sn-110



MAT 5019

Elastic Legendre Coefficients

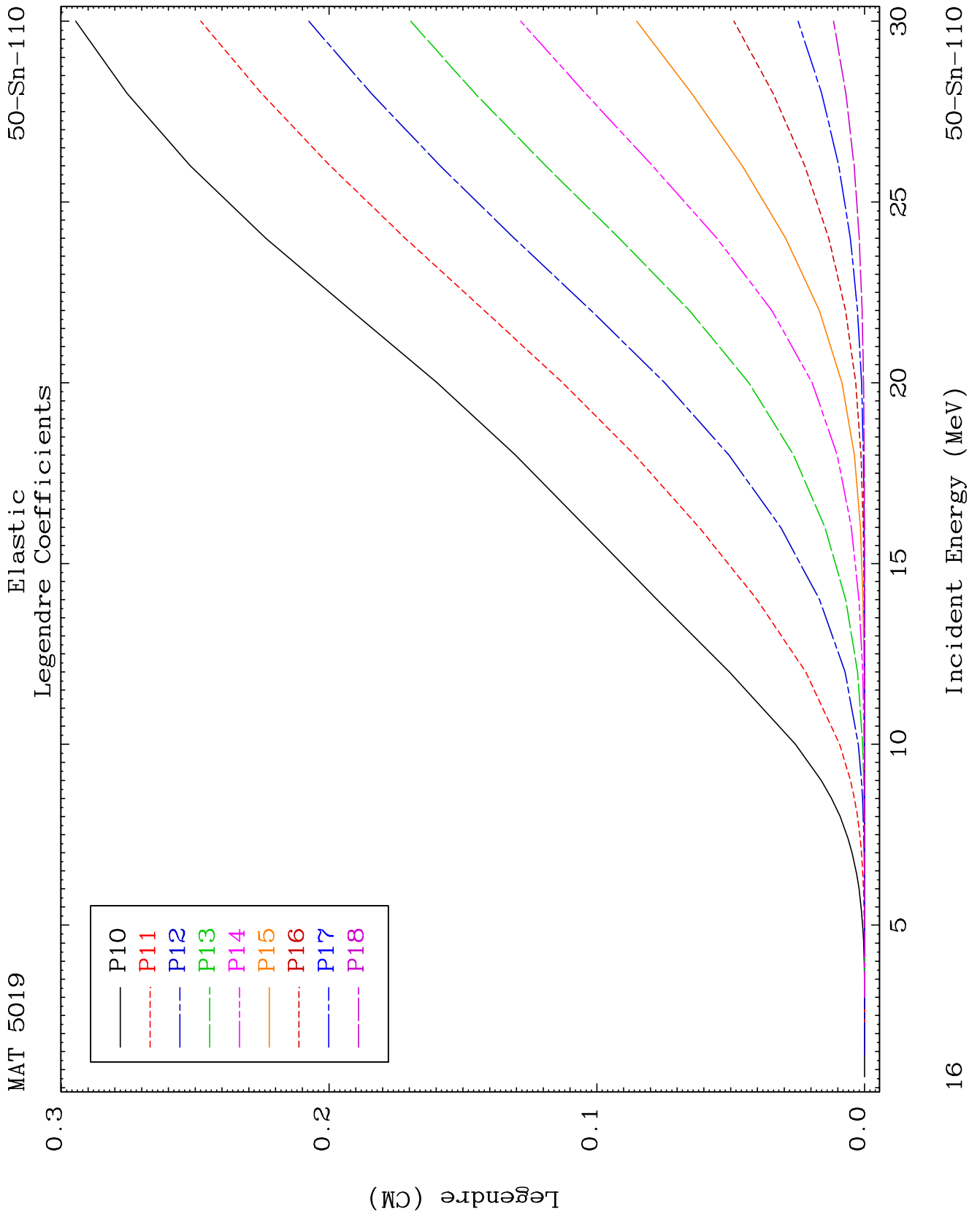
50-Sn-110

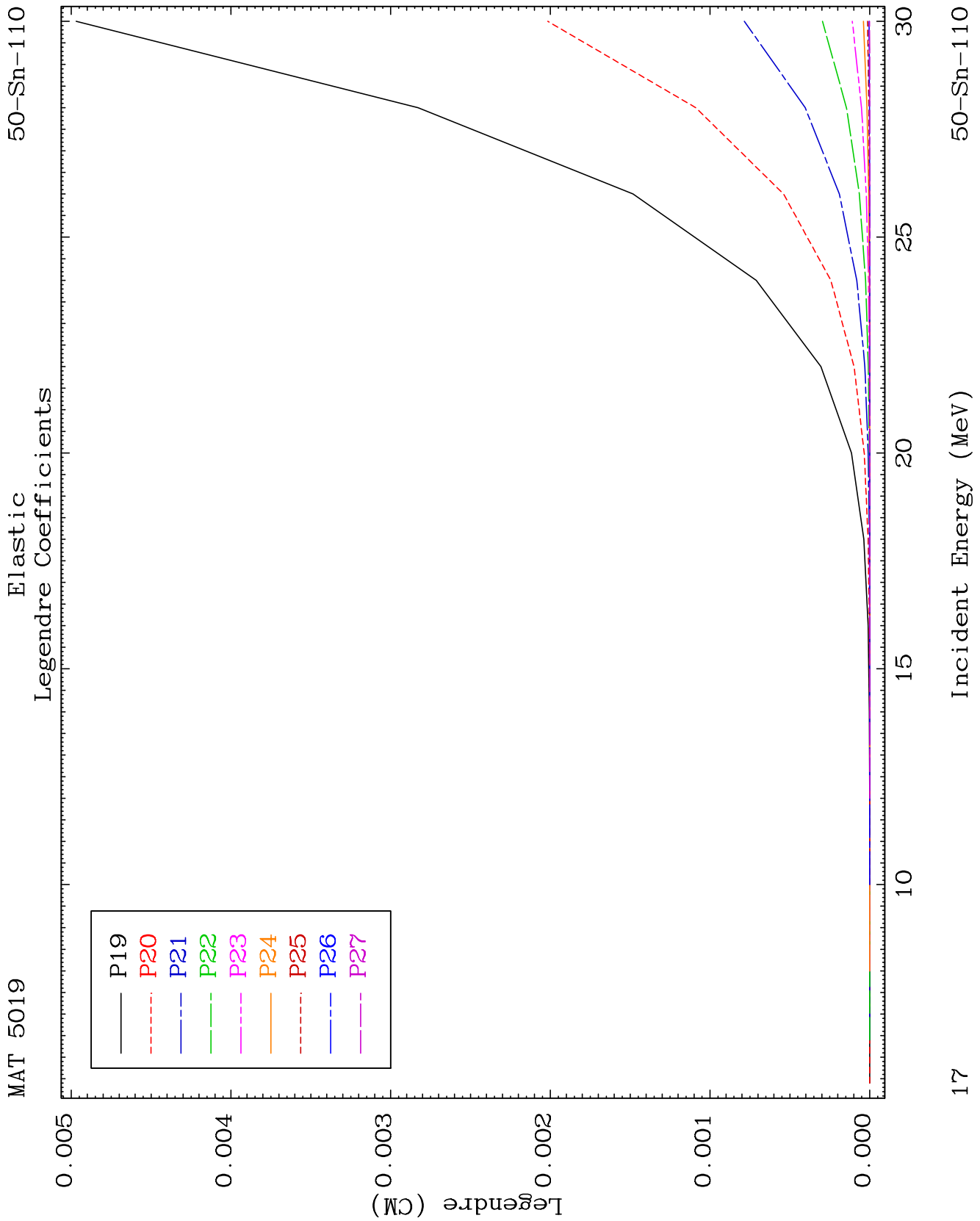


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Incident Energy (MeV)

50-Sn-110

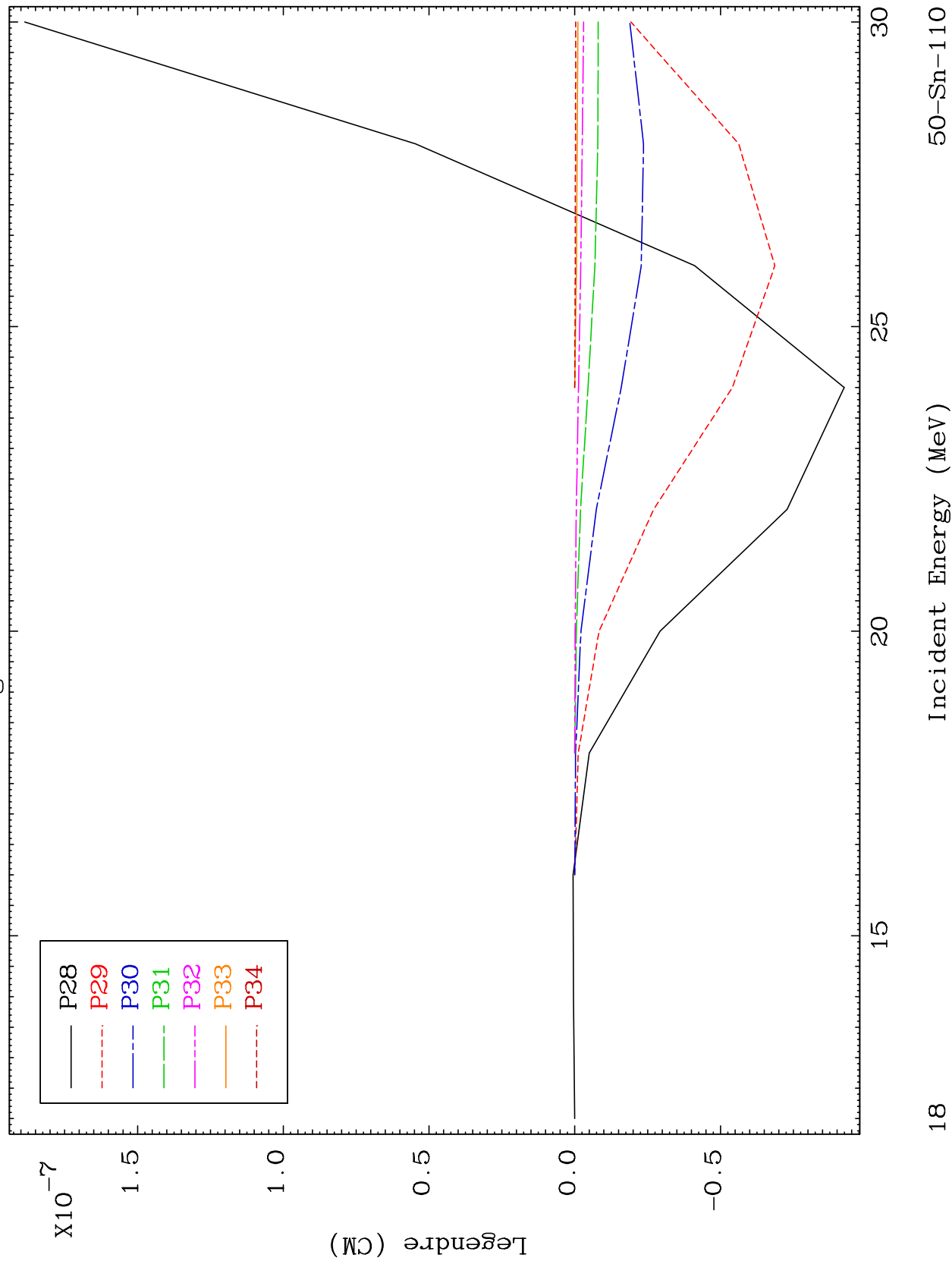




MAT 5019

Elastic Legendre Coefficients

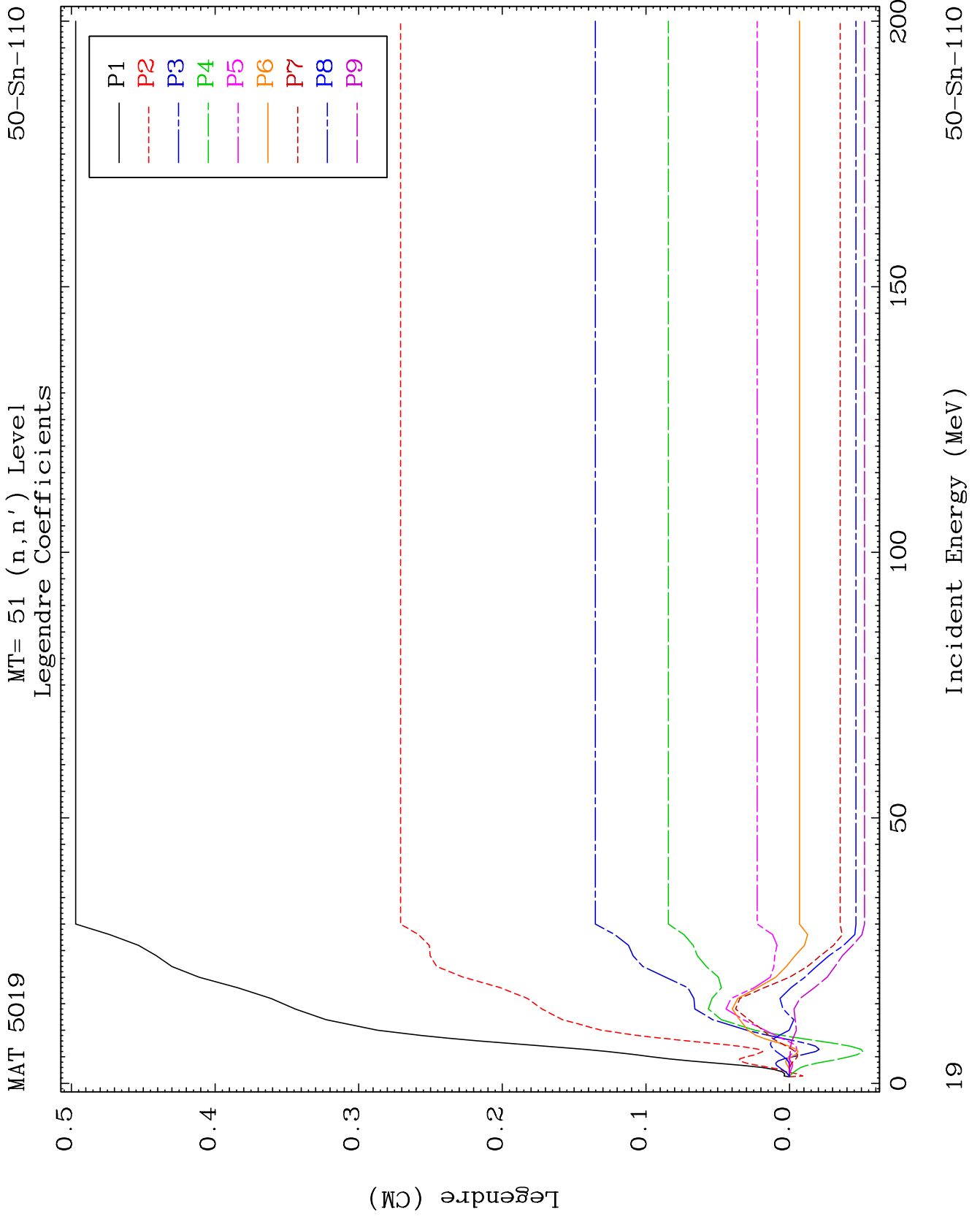
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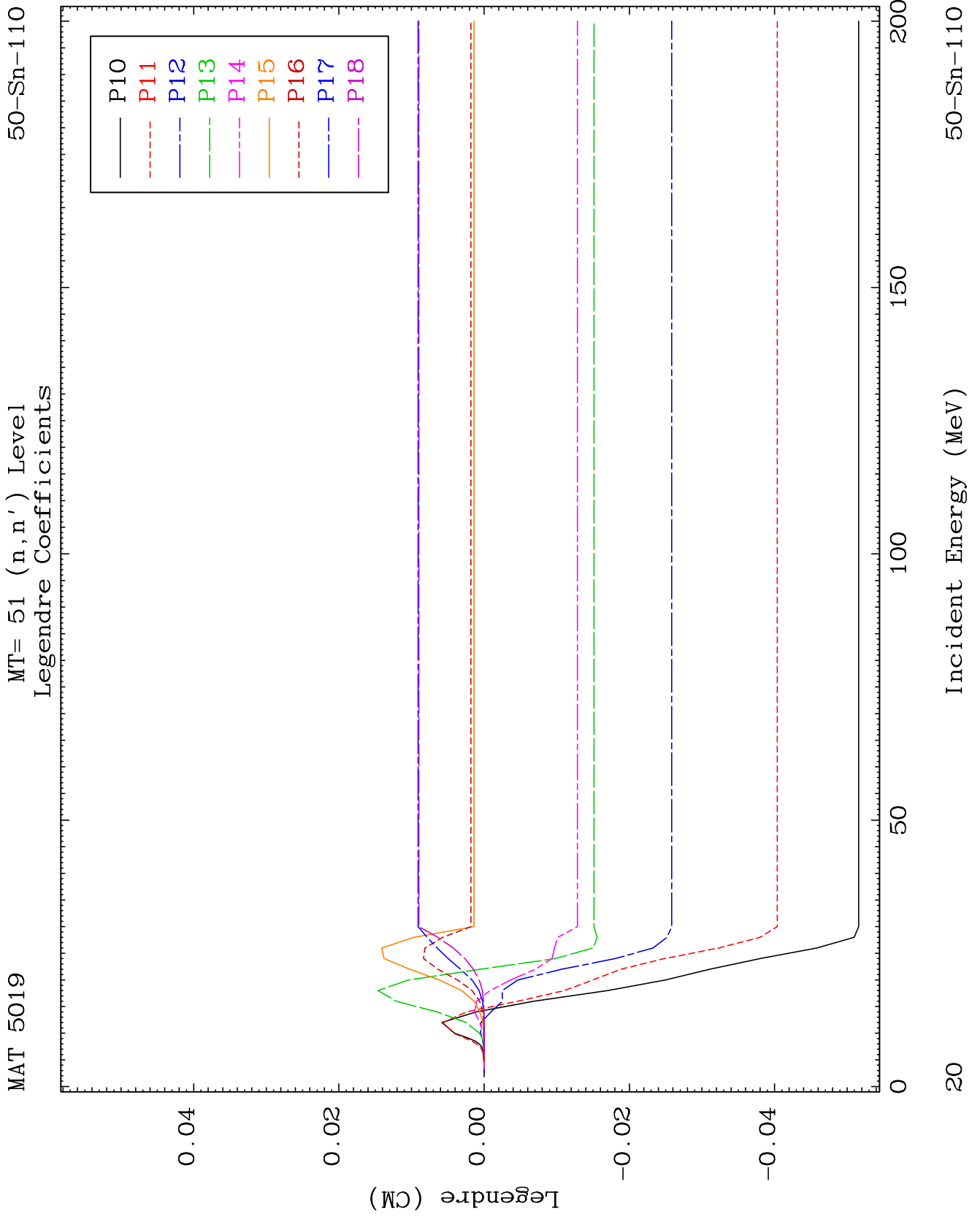


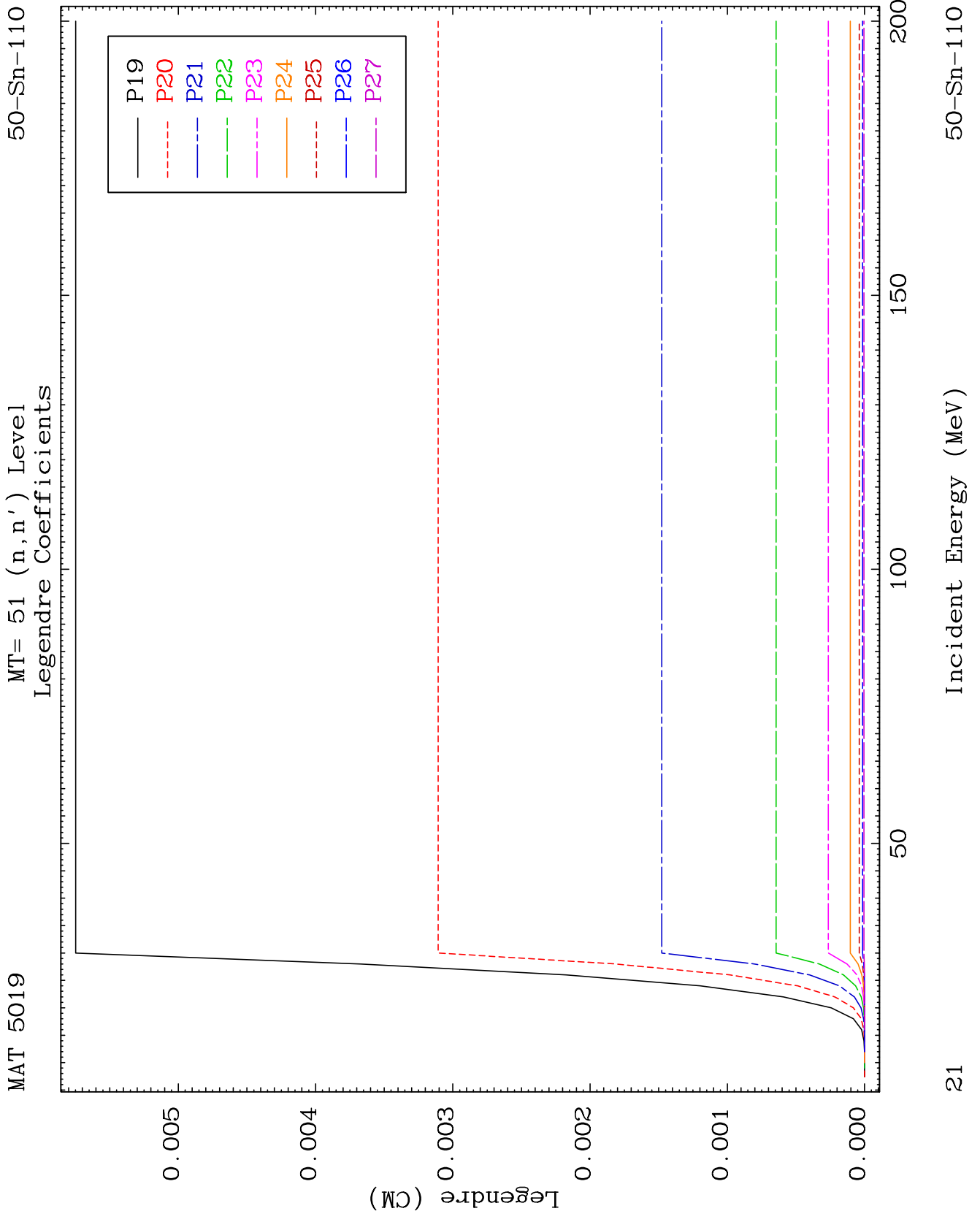
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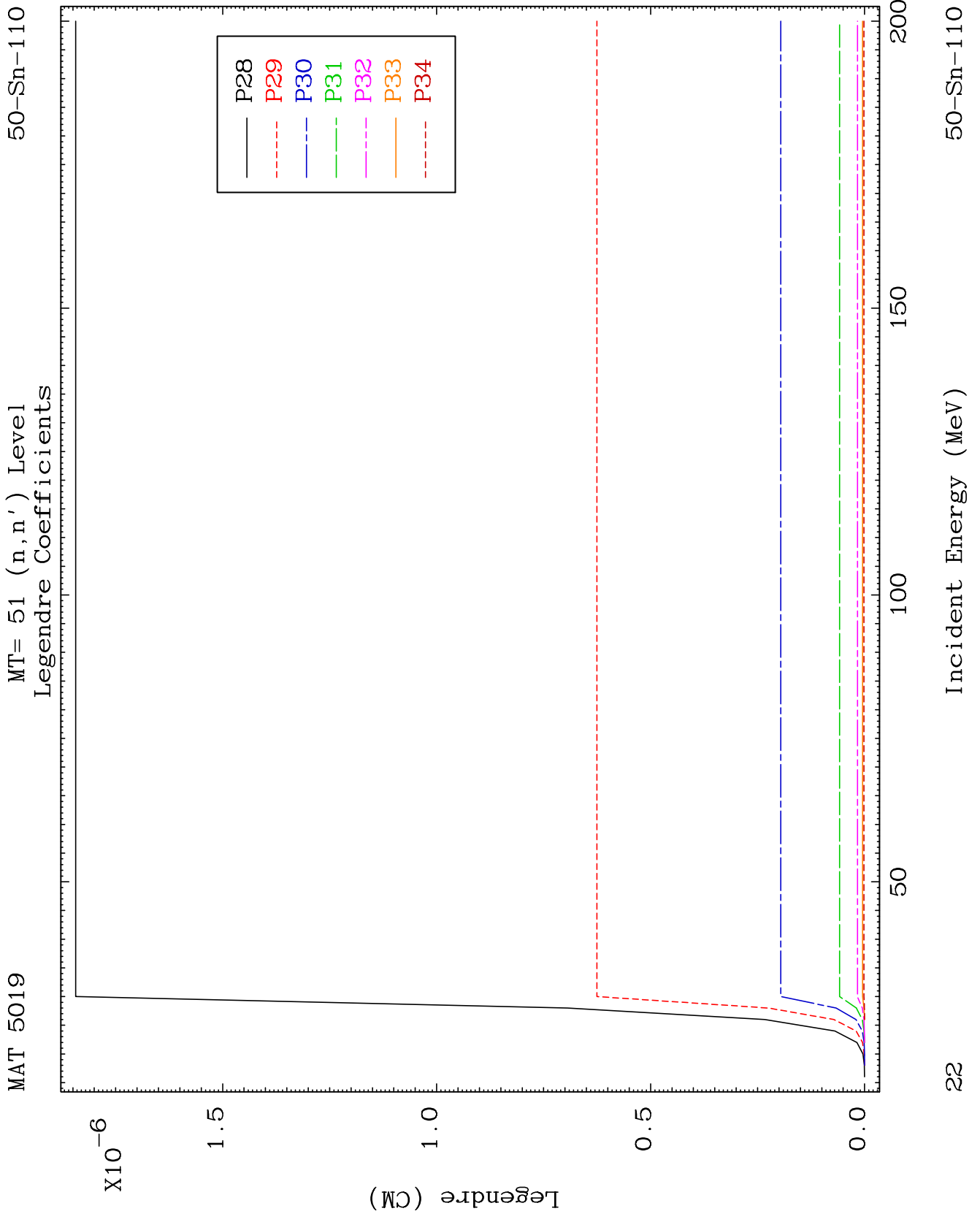
Incident Energy (MeV)

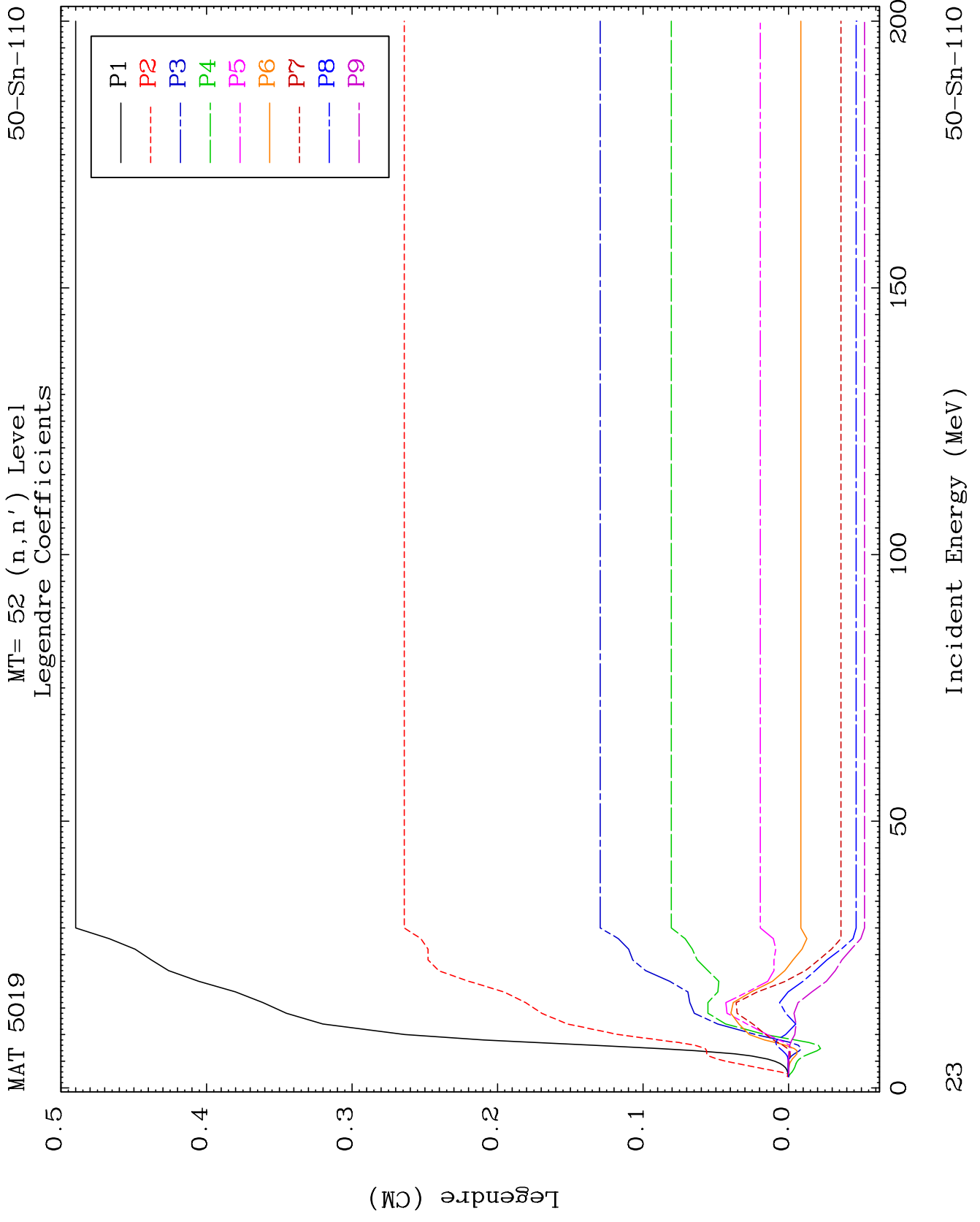
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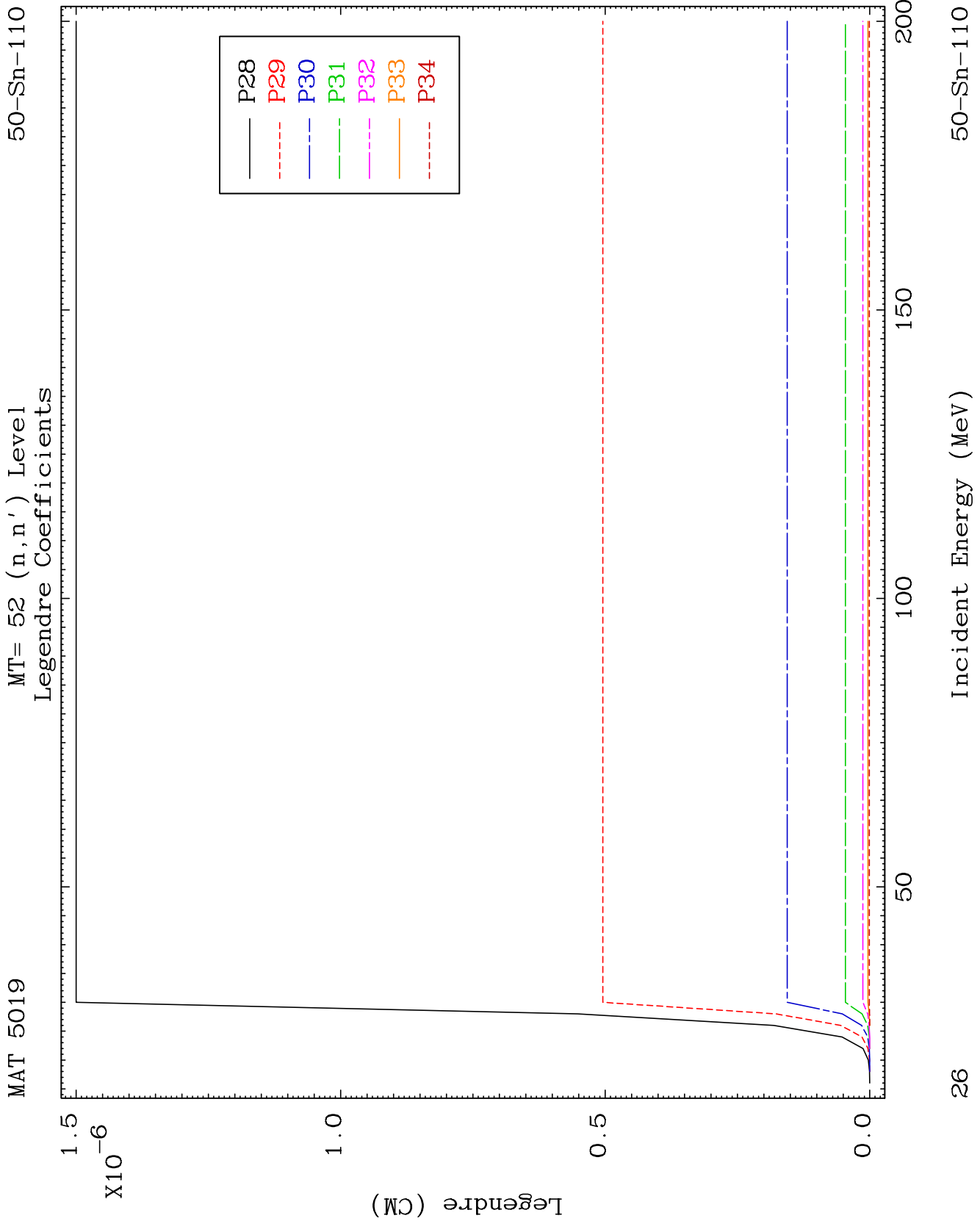


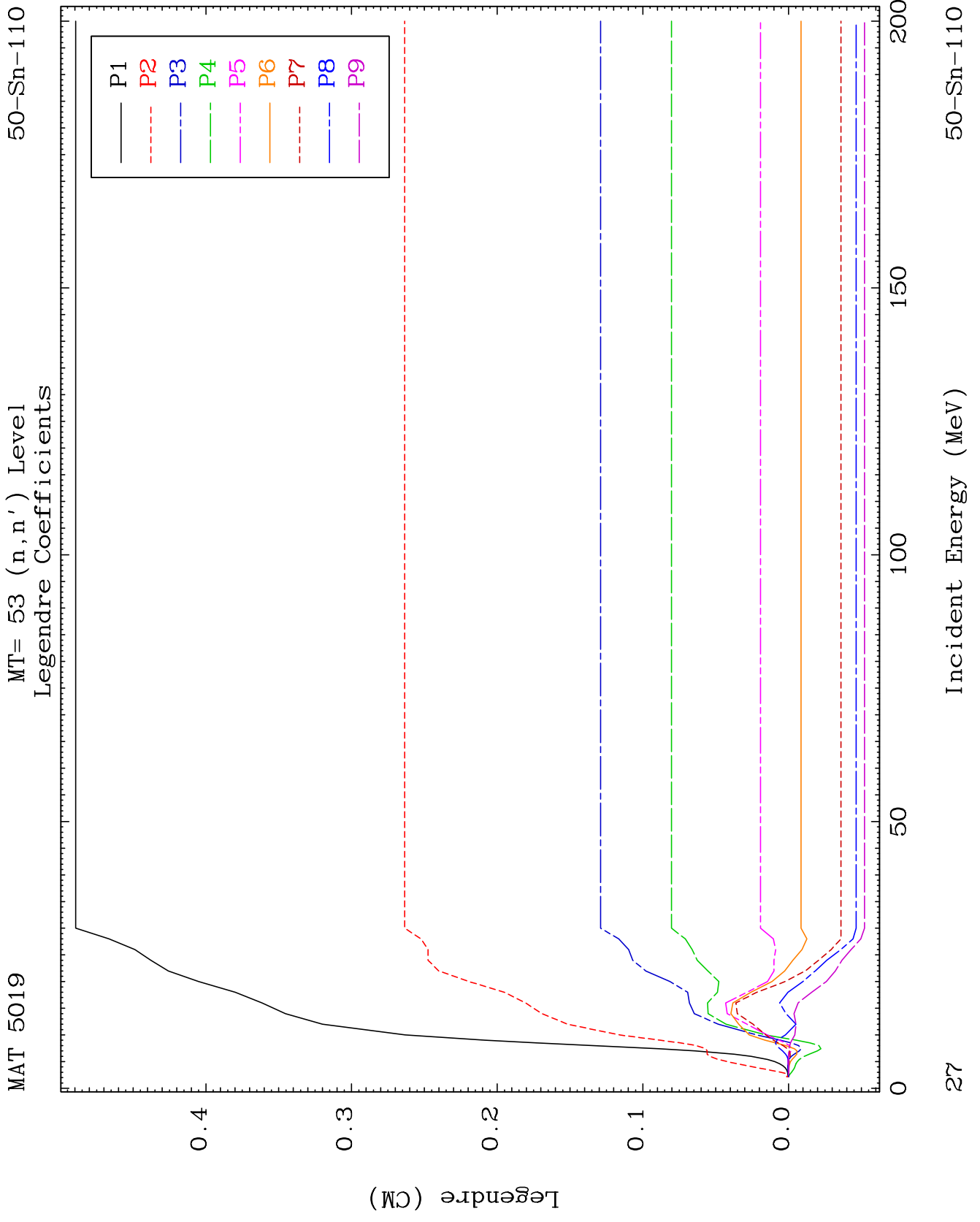


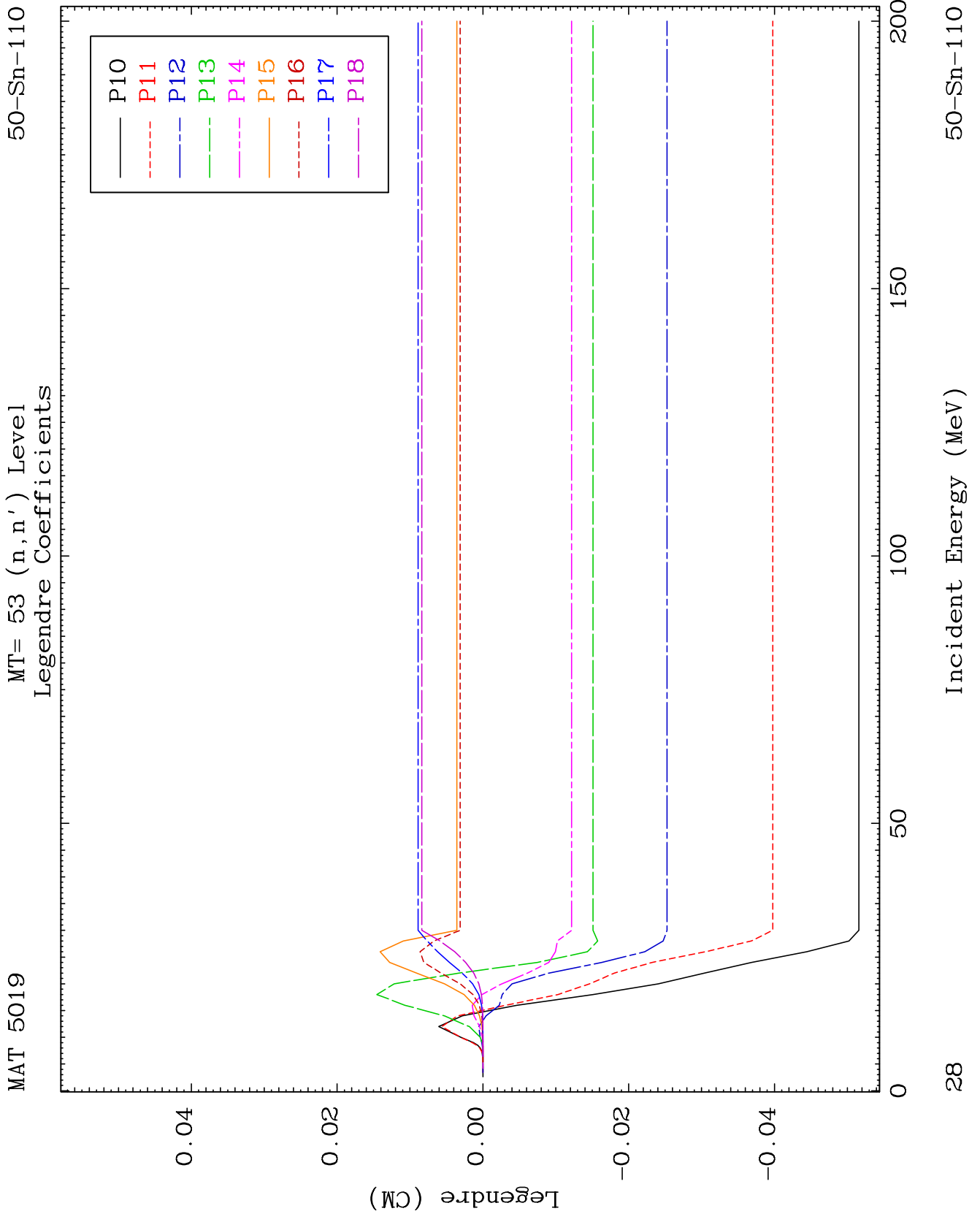


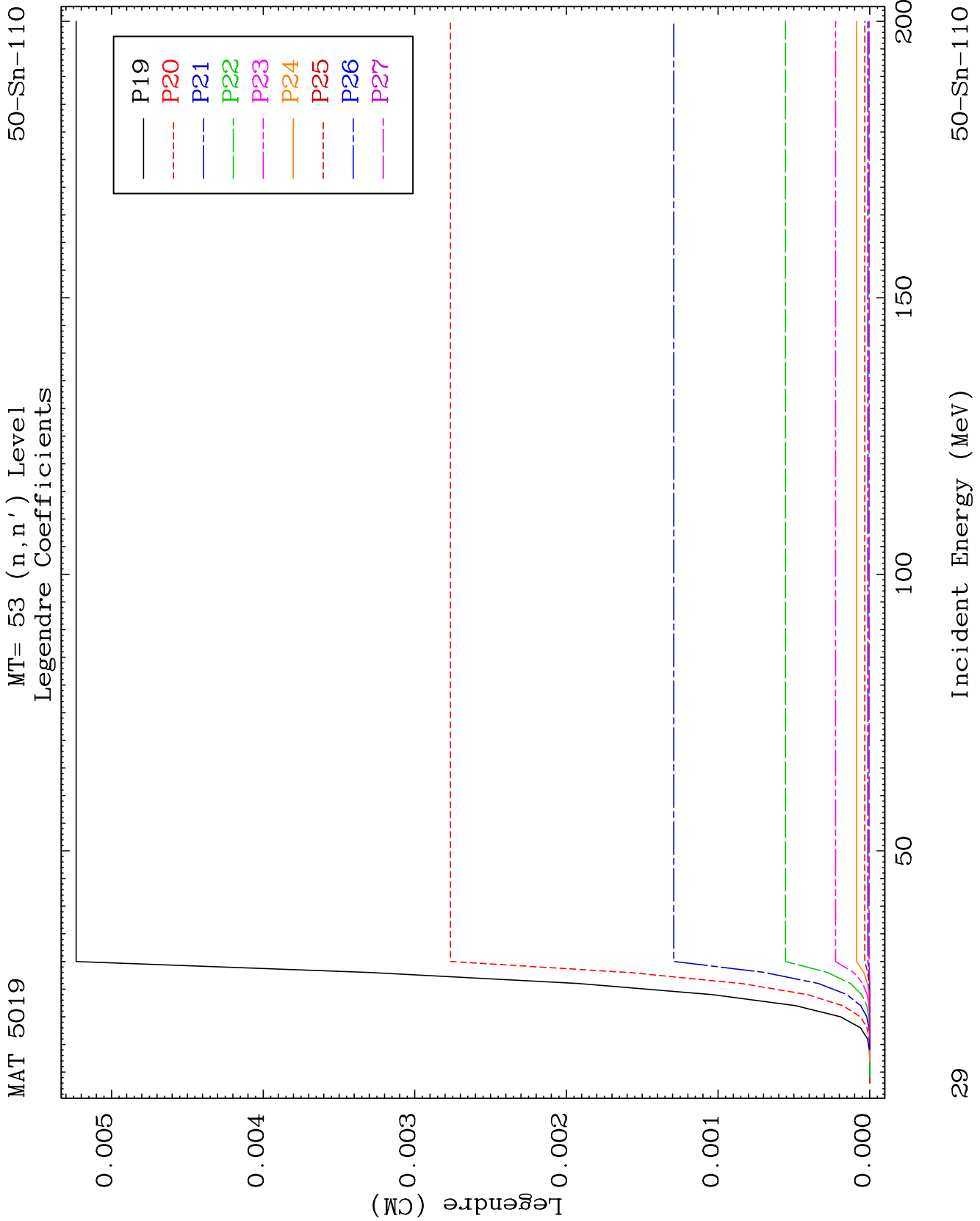


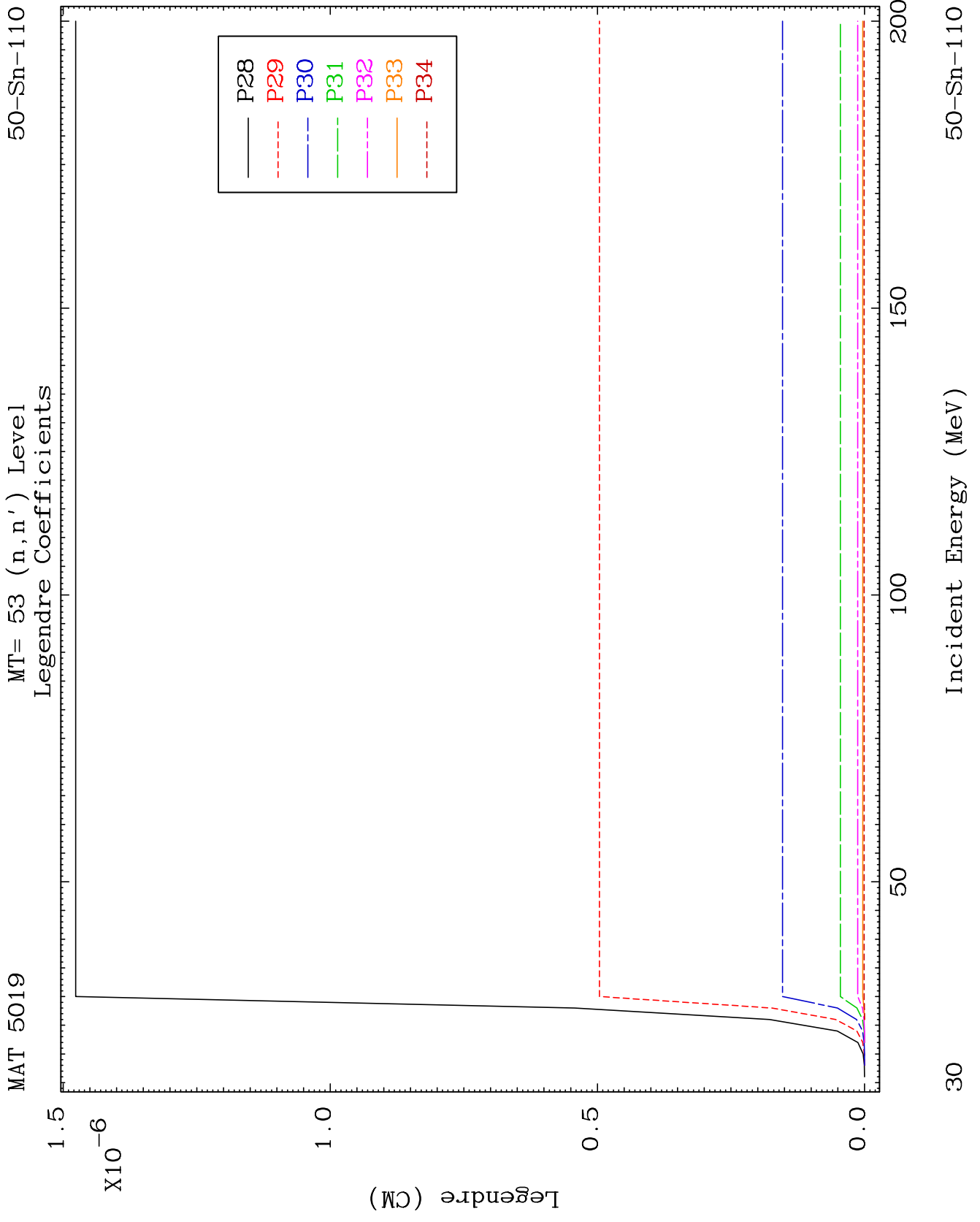








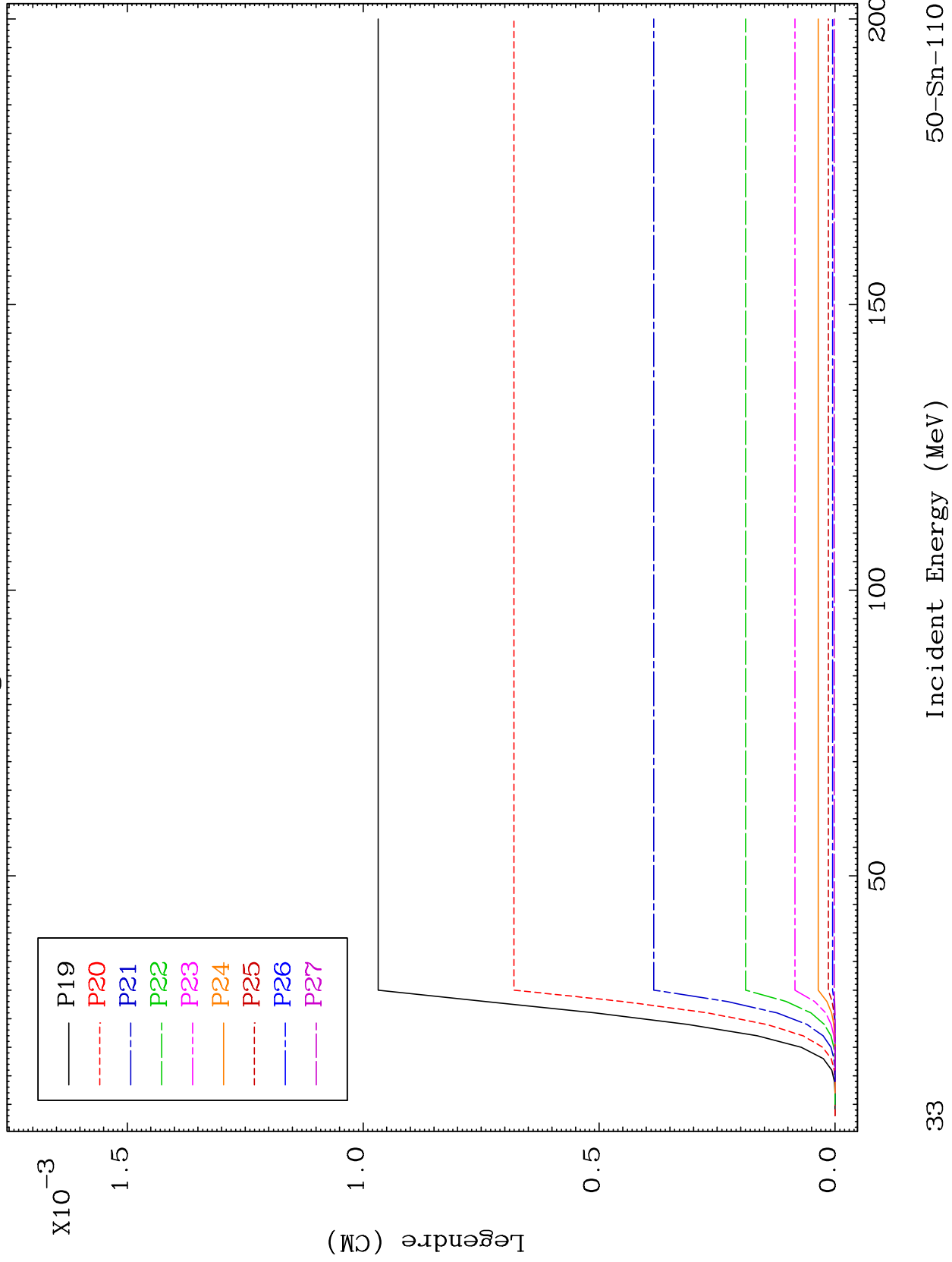


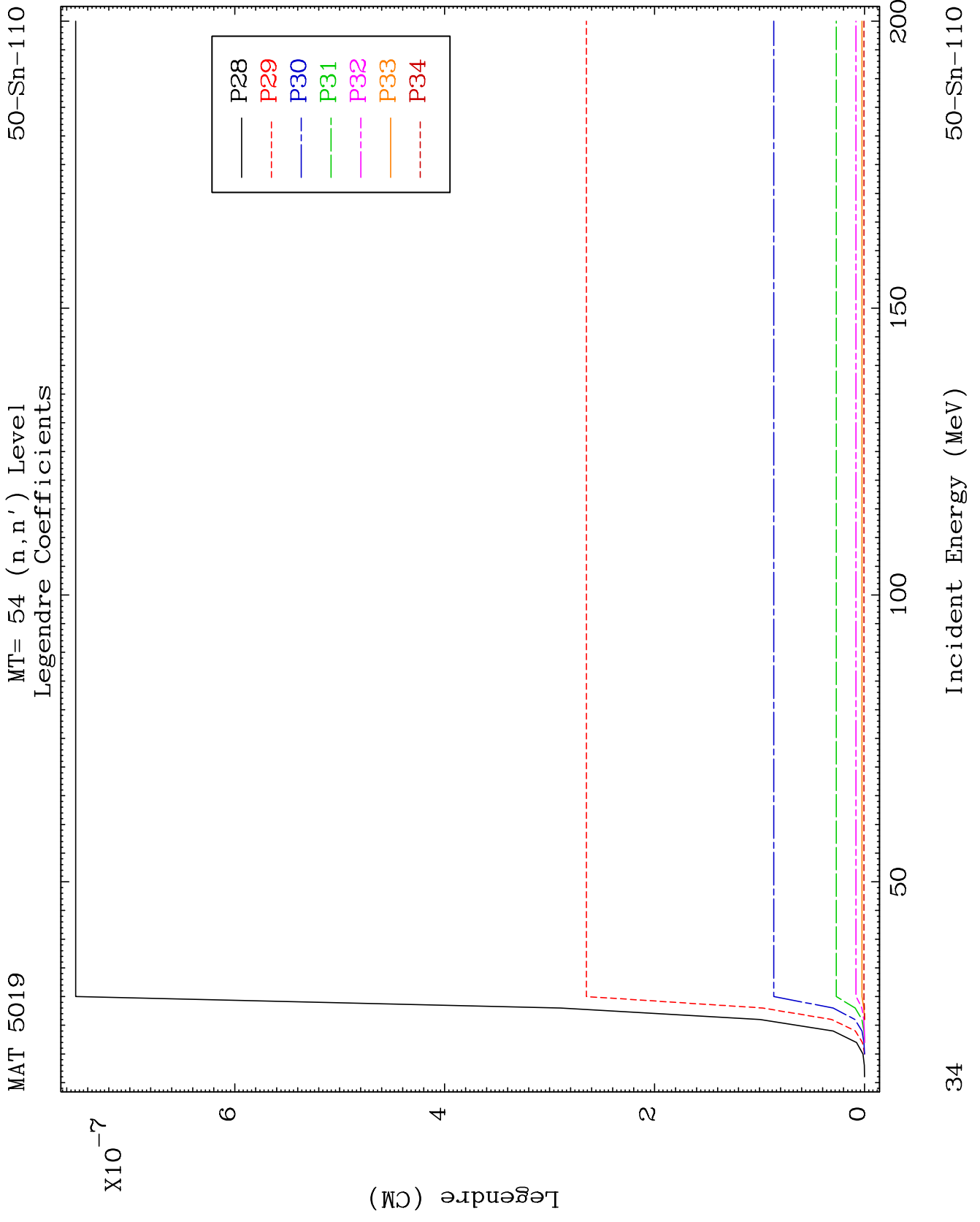


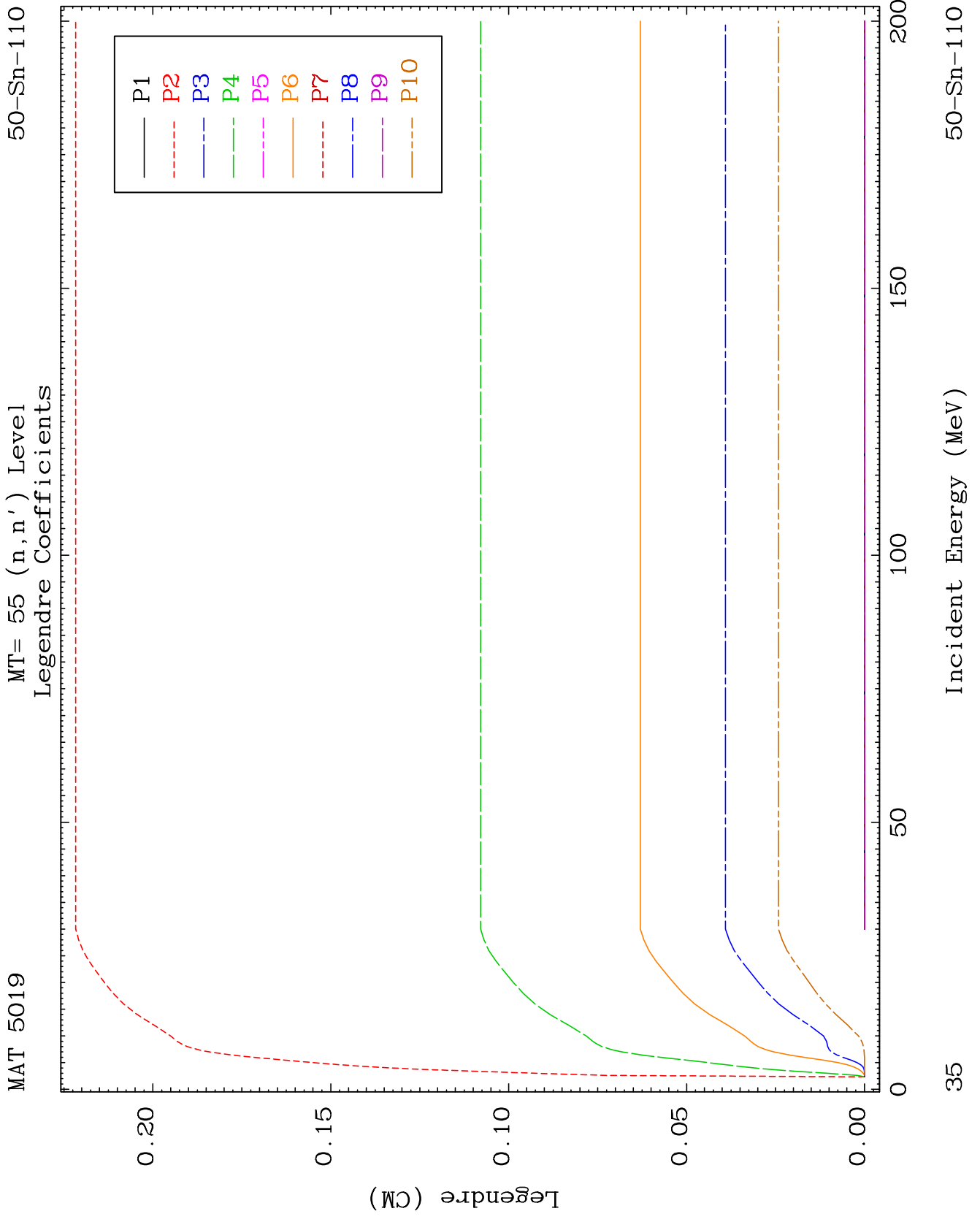
MAT 5019

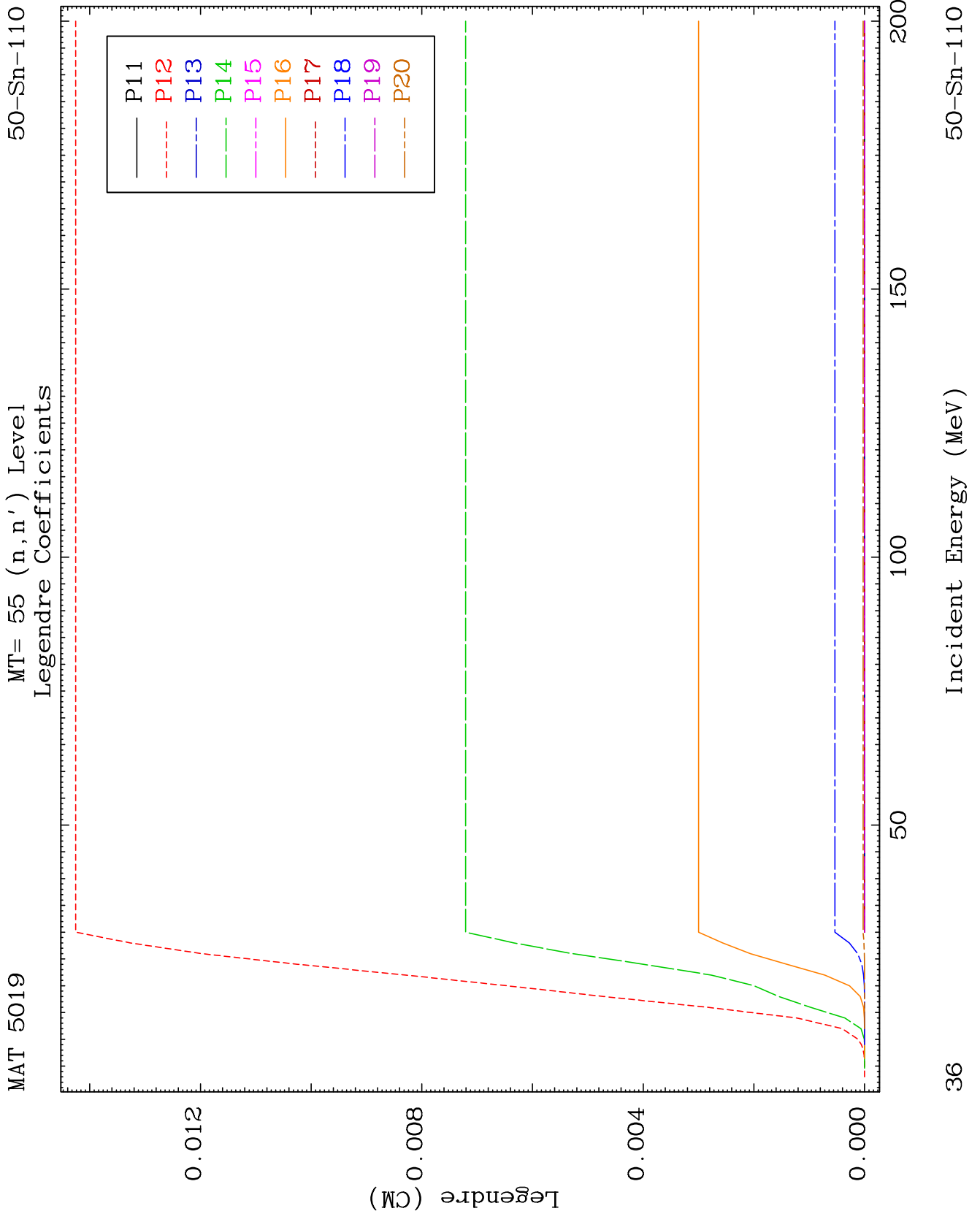
MT= 54 (n,n') Level
Legendre Coefficients

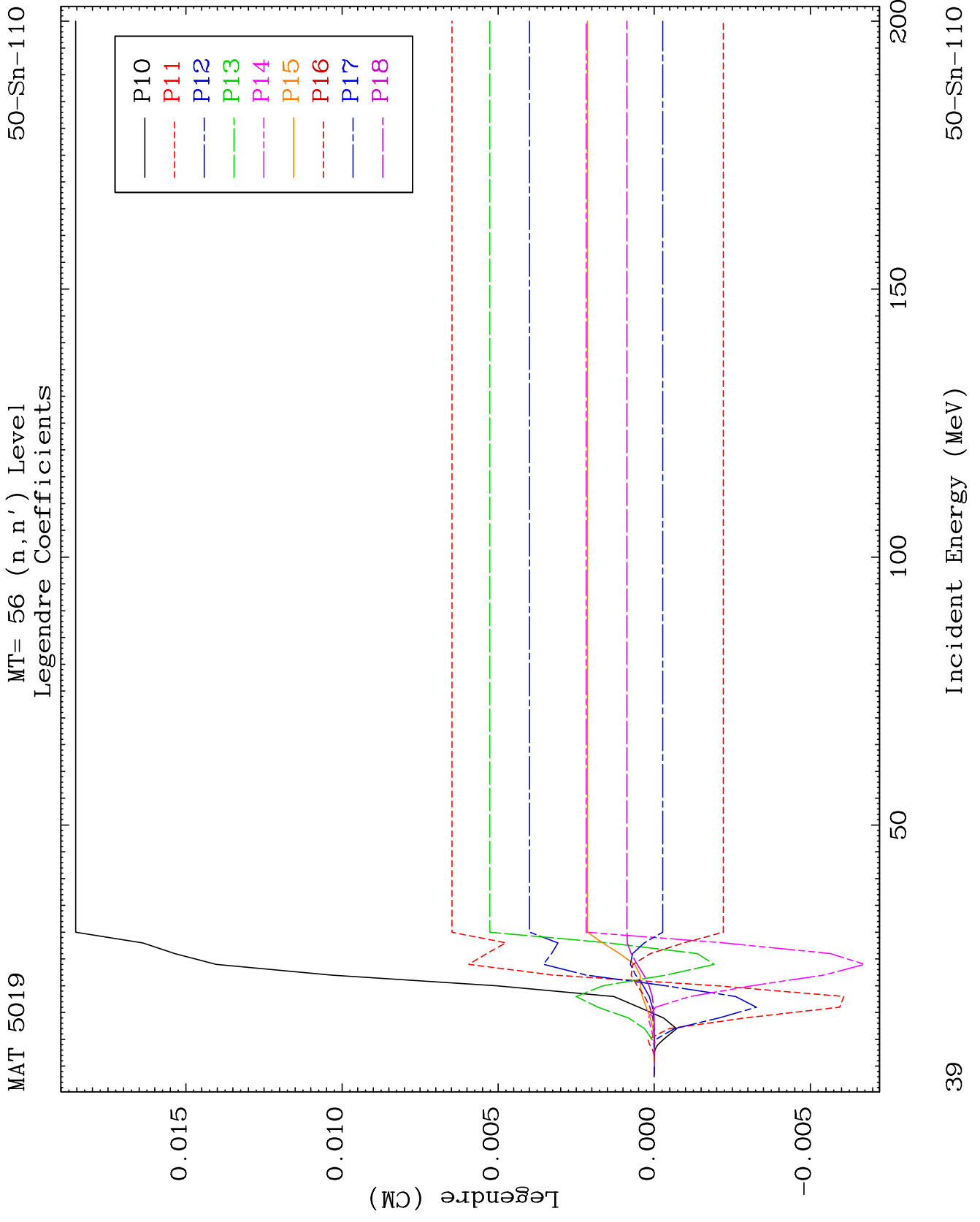
50-Sn-110







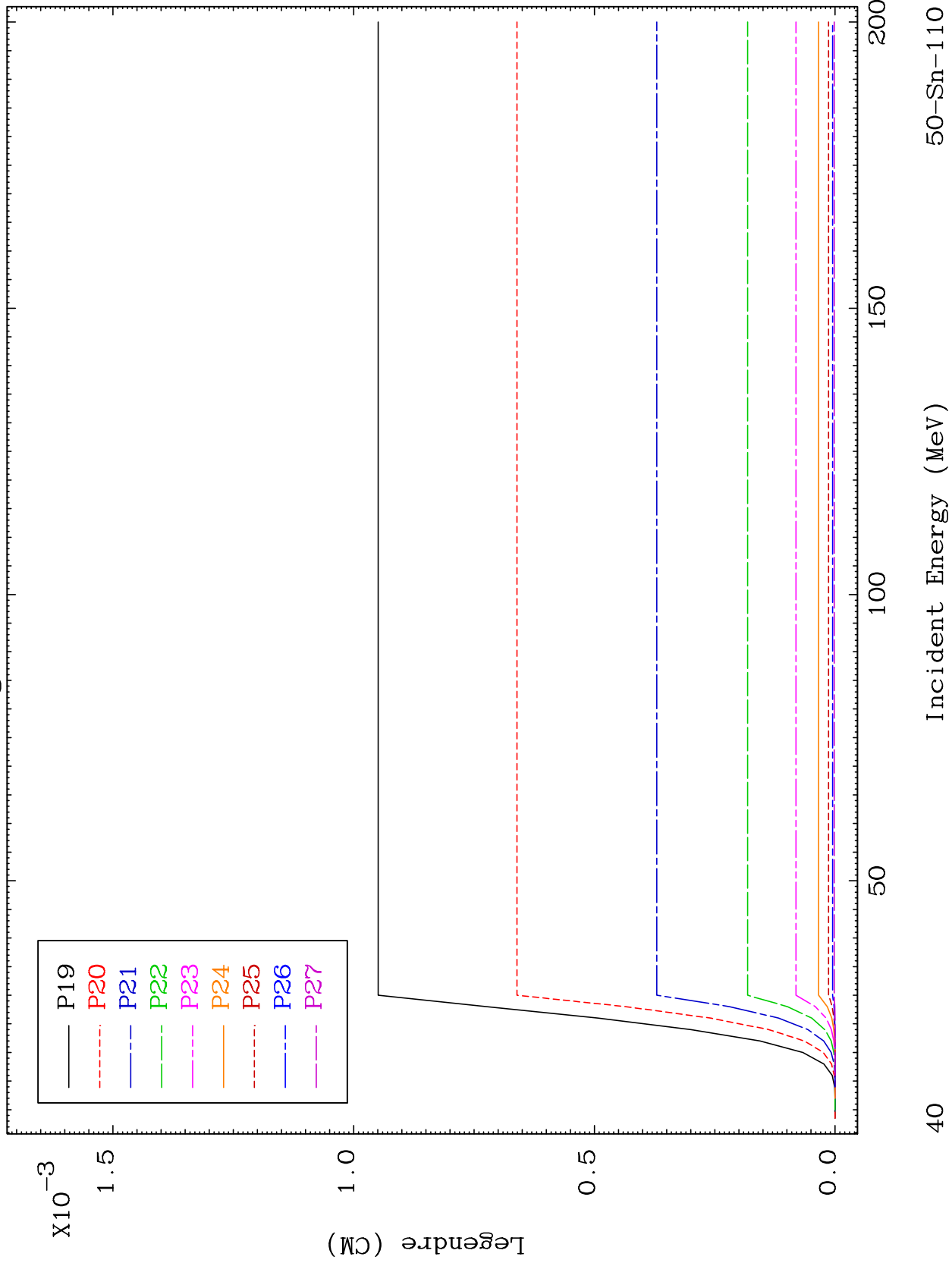




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MT= 56 (n,n') Level
Legendre Coefficients

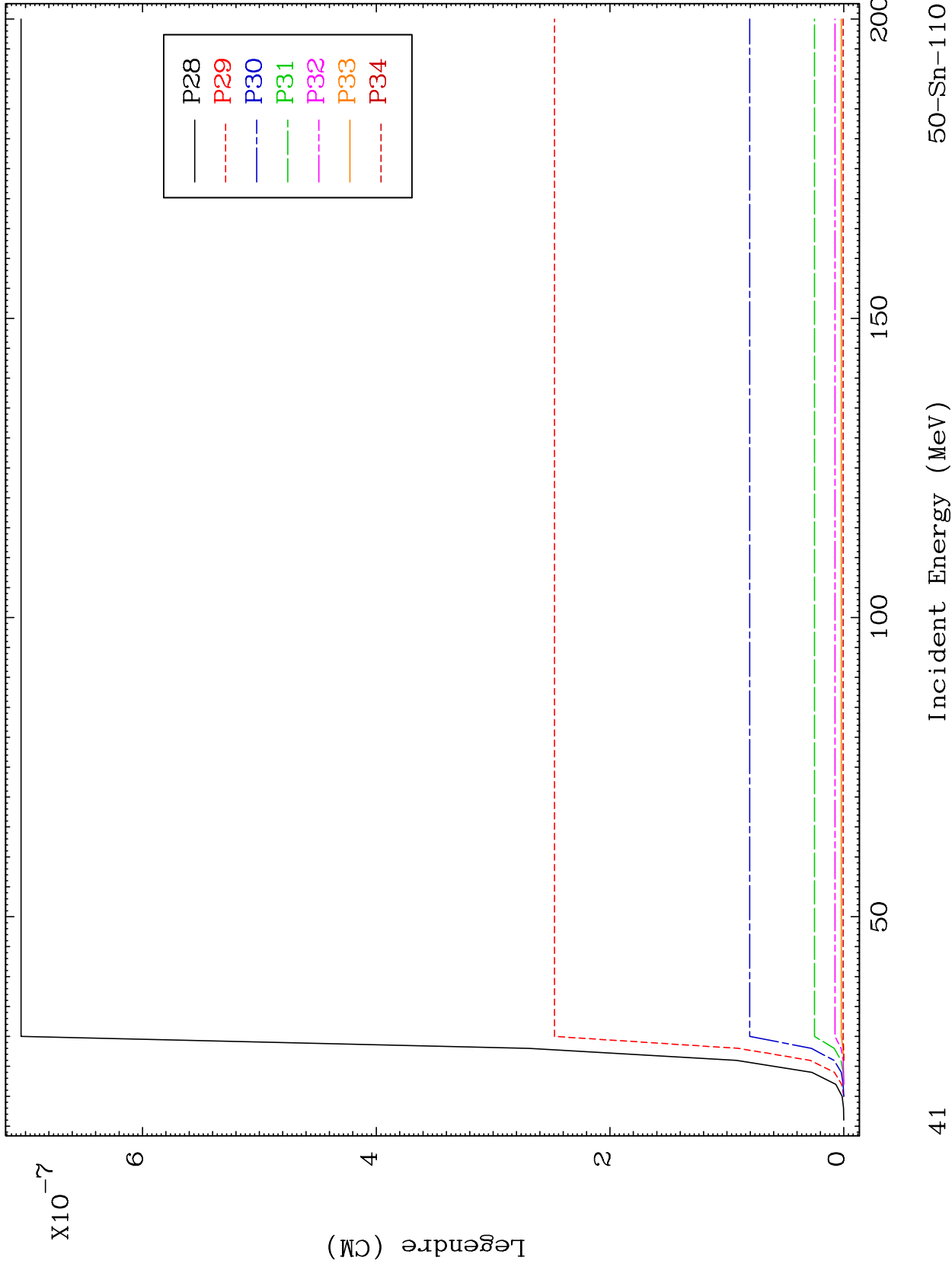
50-Sn-110



MAT 5019

MT= 56 (n,n') Level
Legendre Coefficients

50-Sn-110



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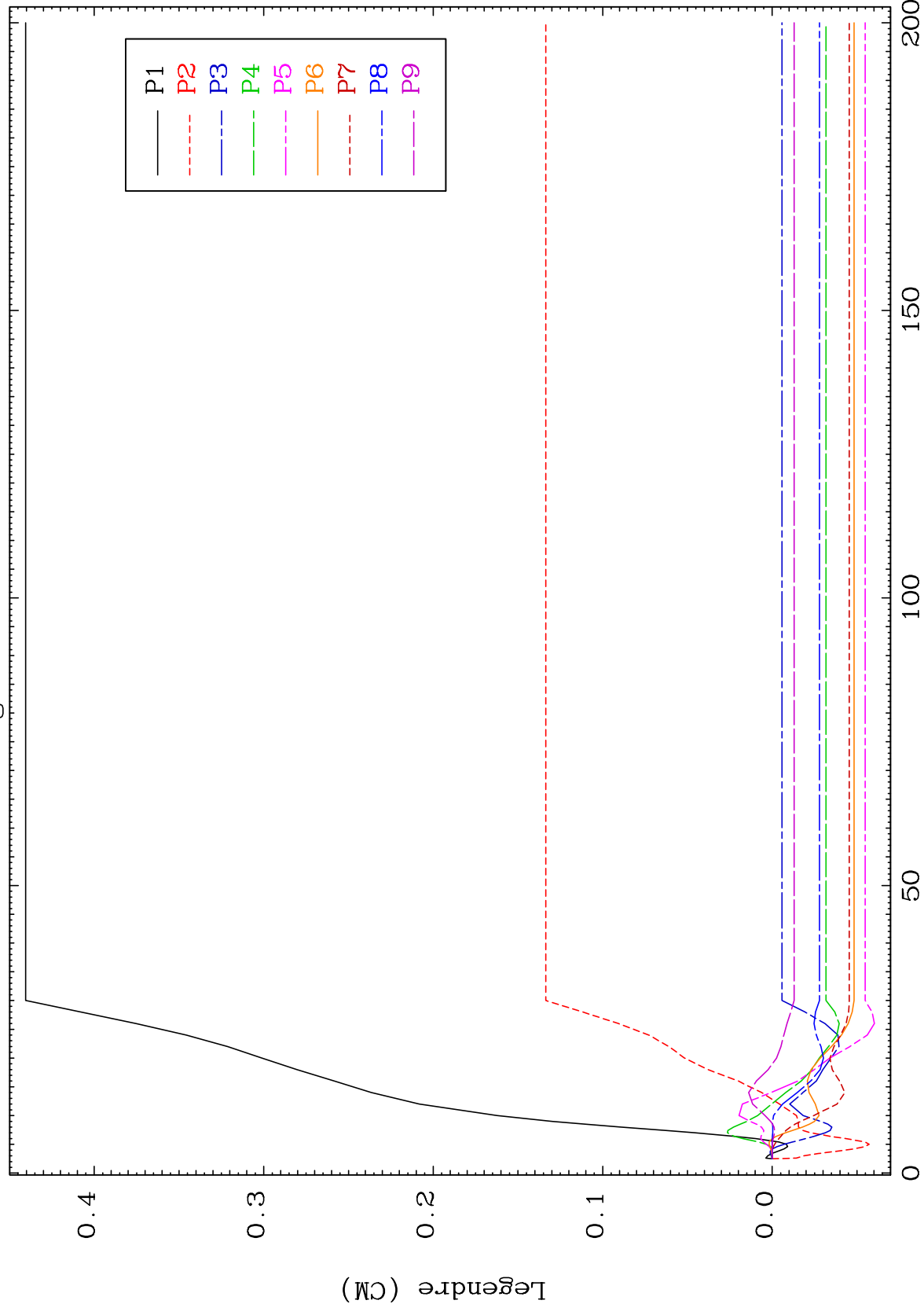
Incident Energy (MeV)

50-Sn-110

MAT 5019

MT= 57 (n,n') Level
Legendre Coefficients

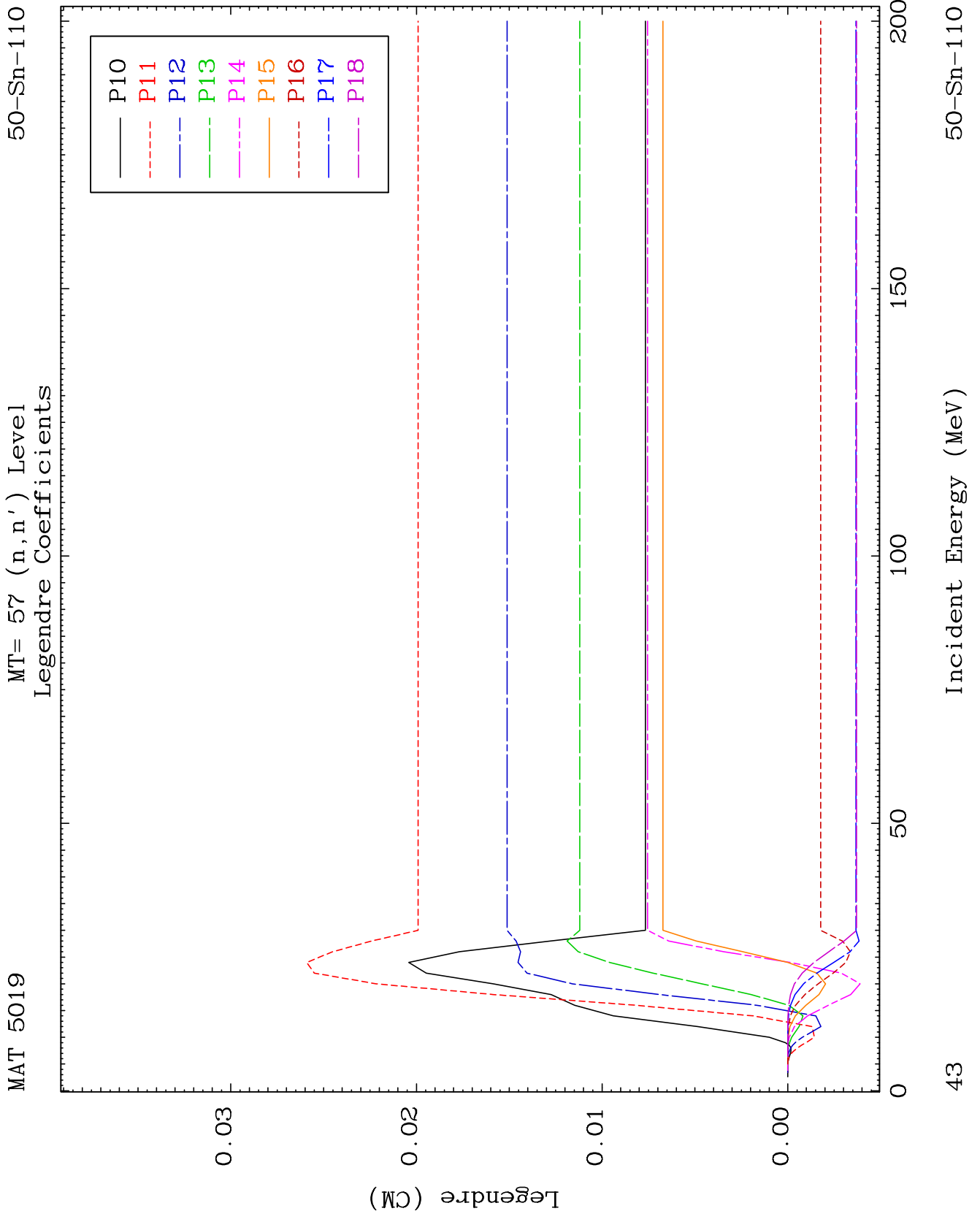
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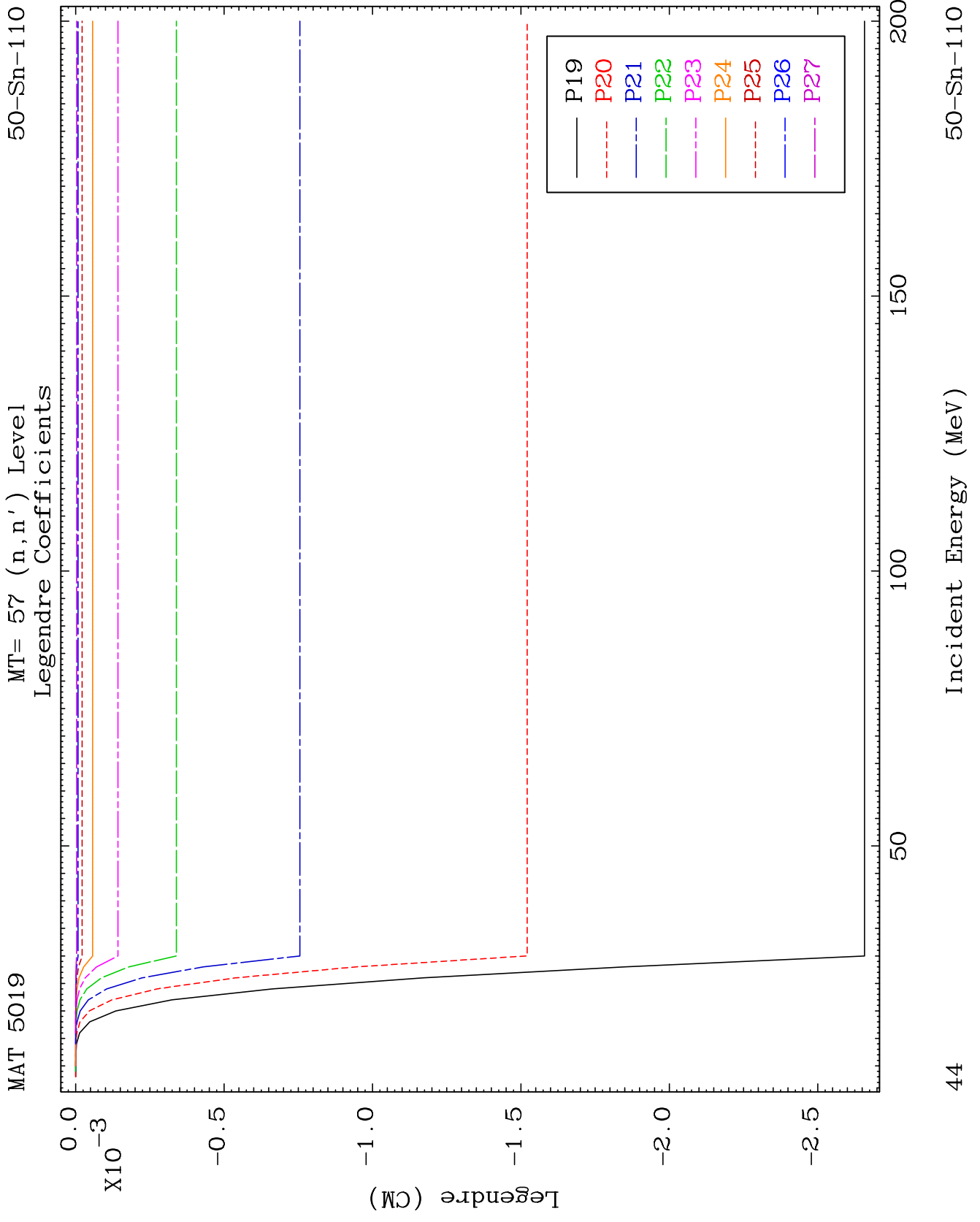


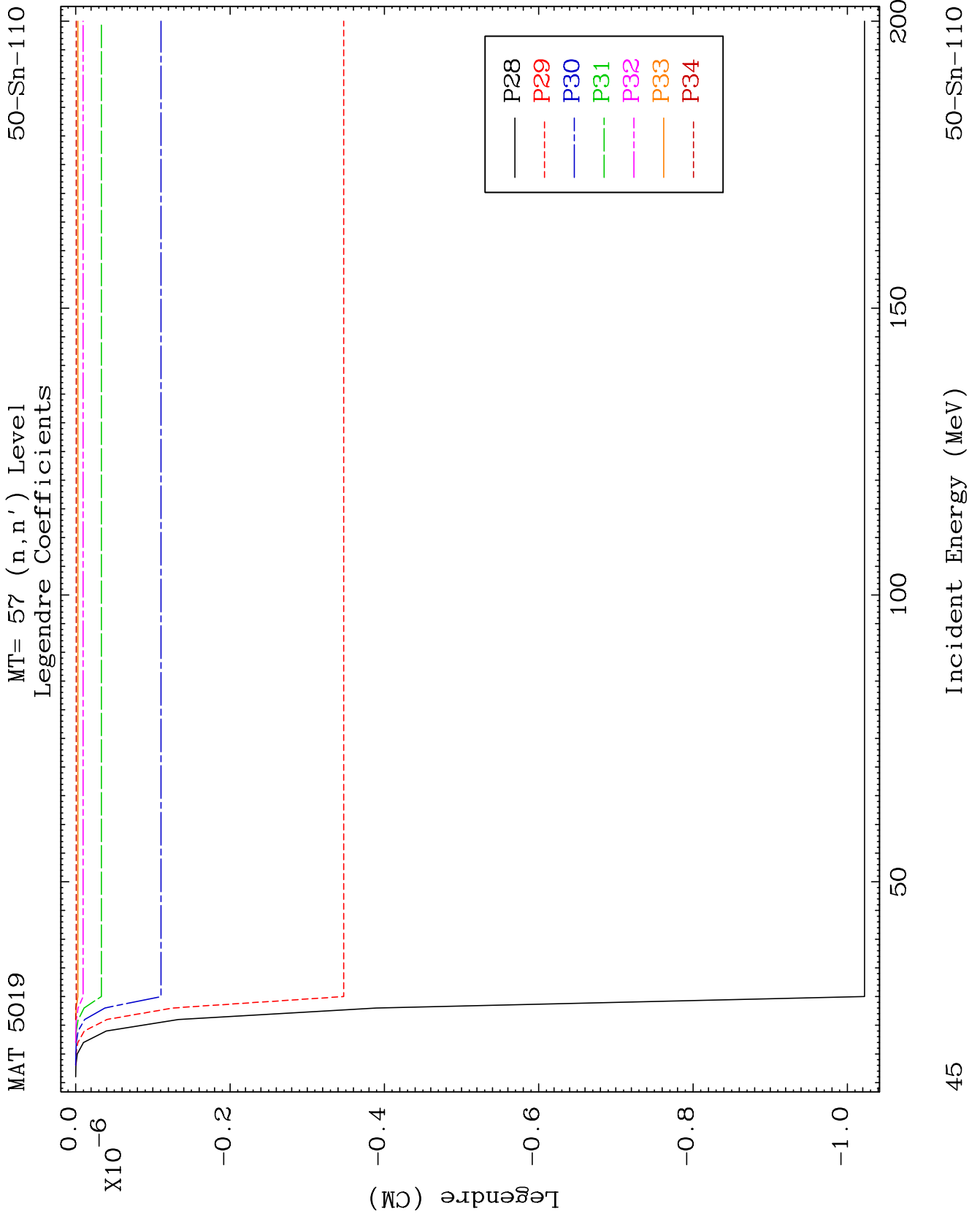
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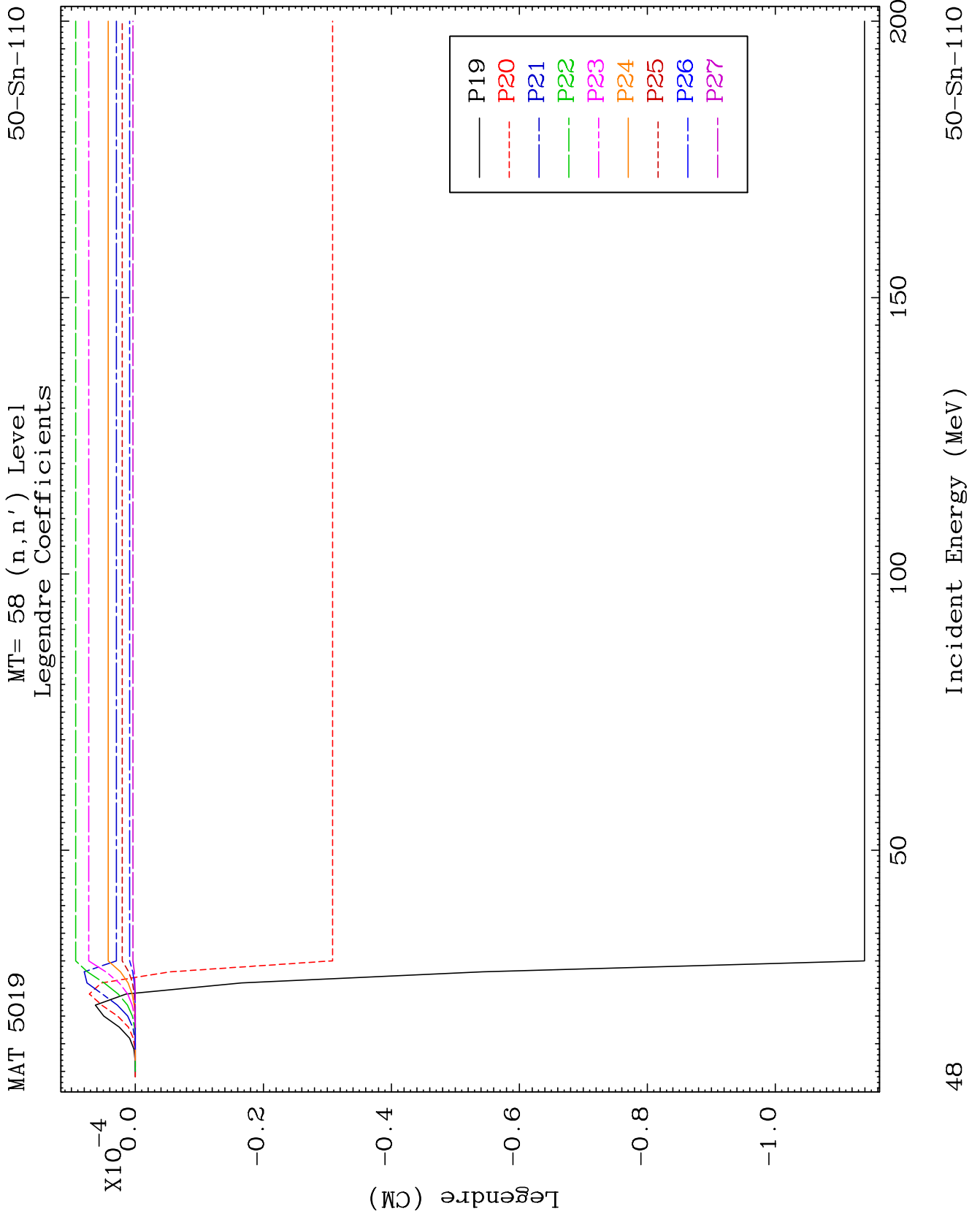
Incident Energy (MeV)

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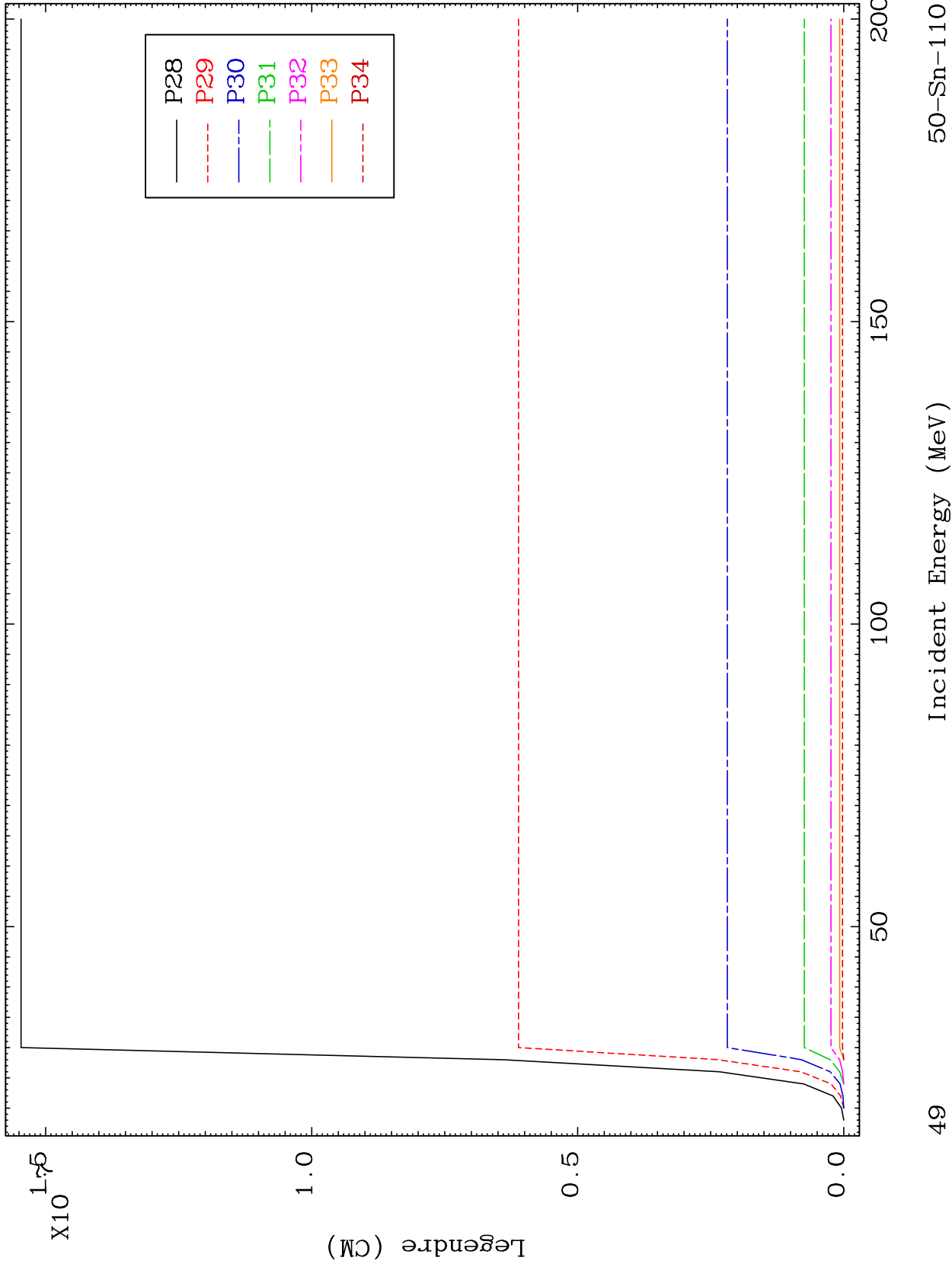


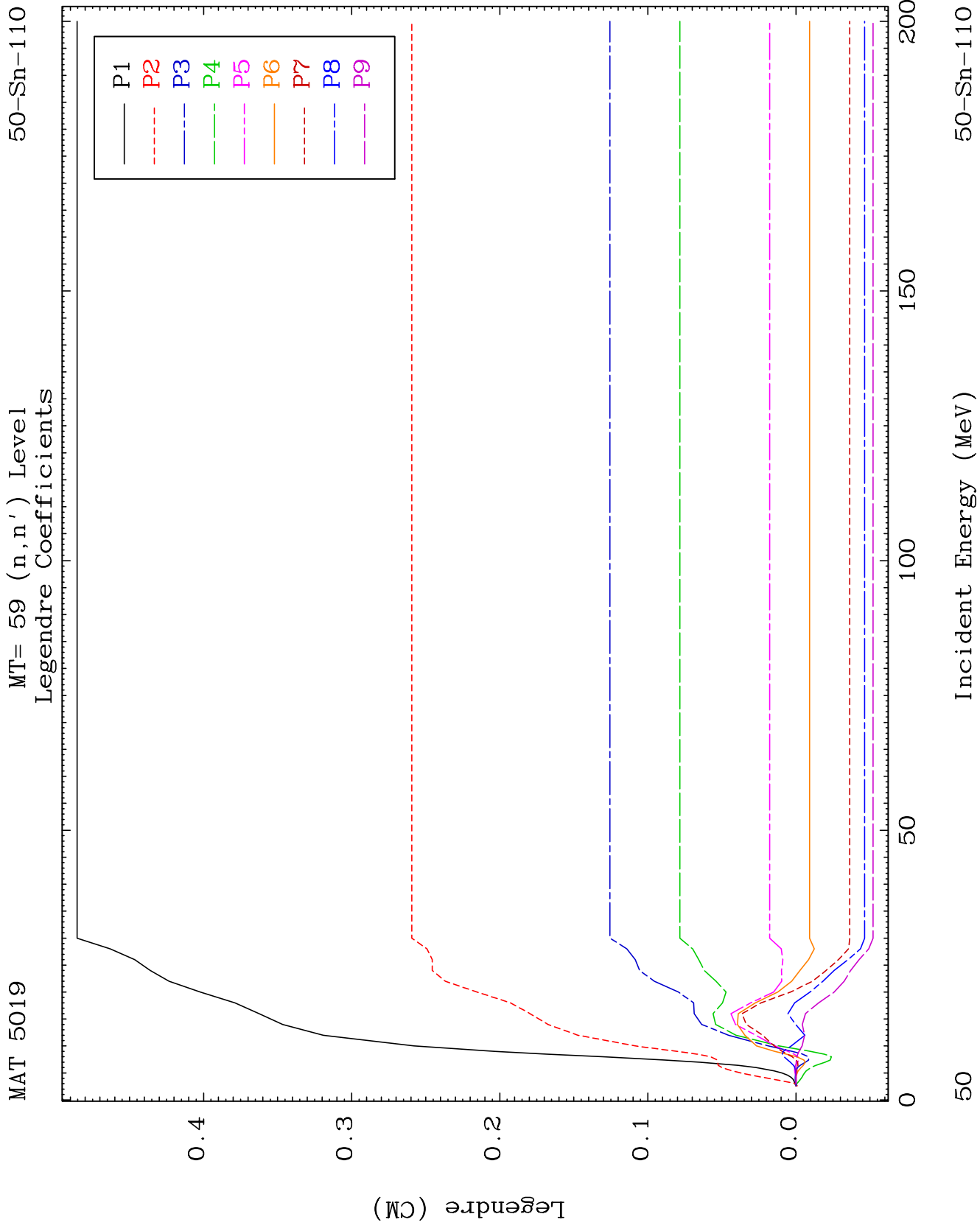


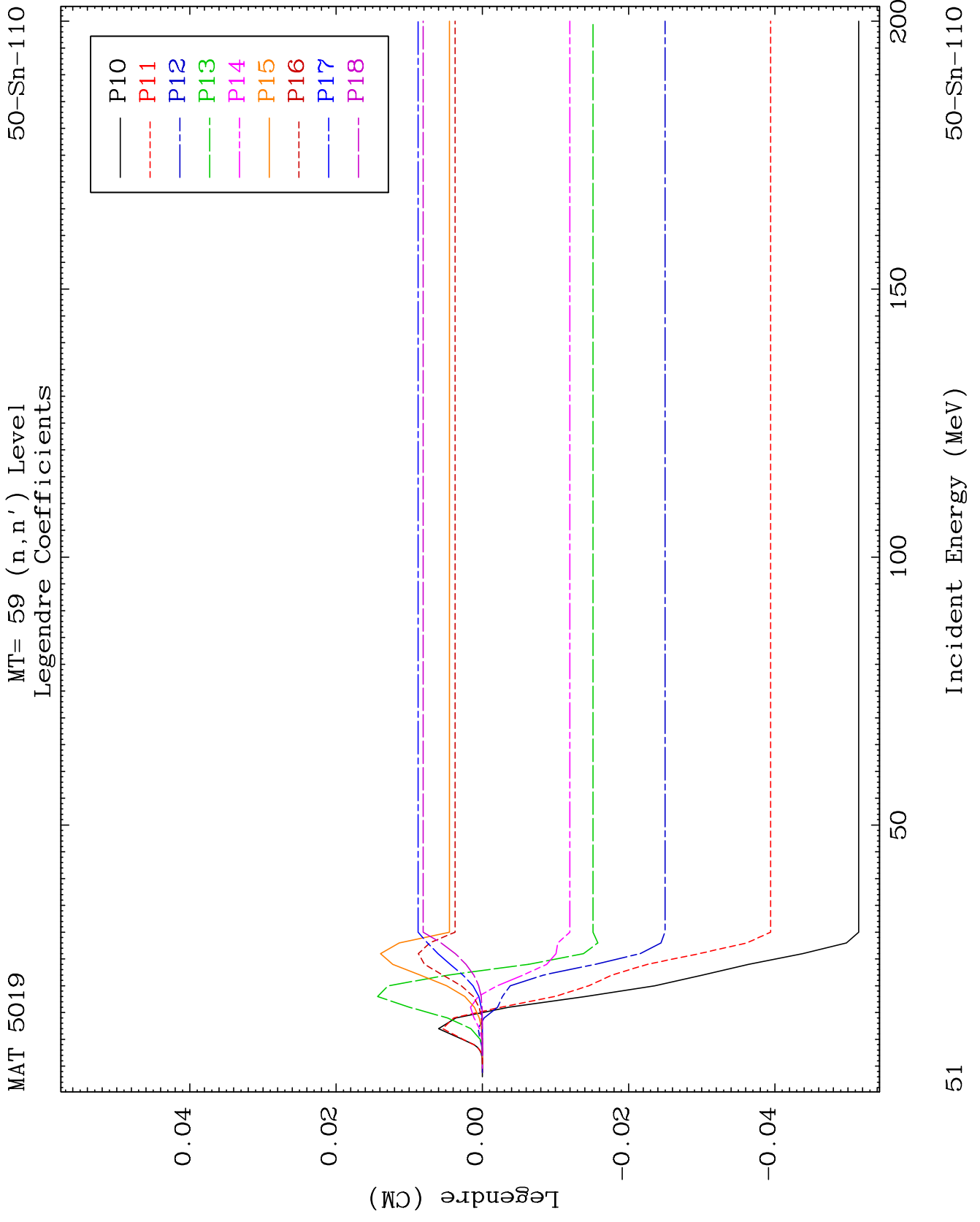
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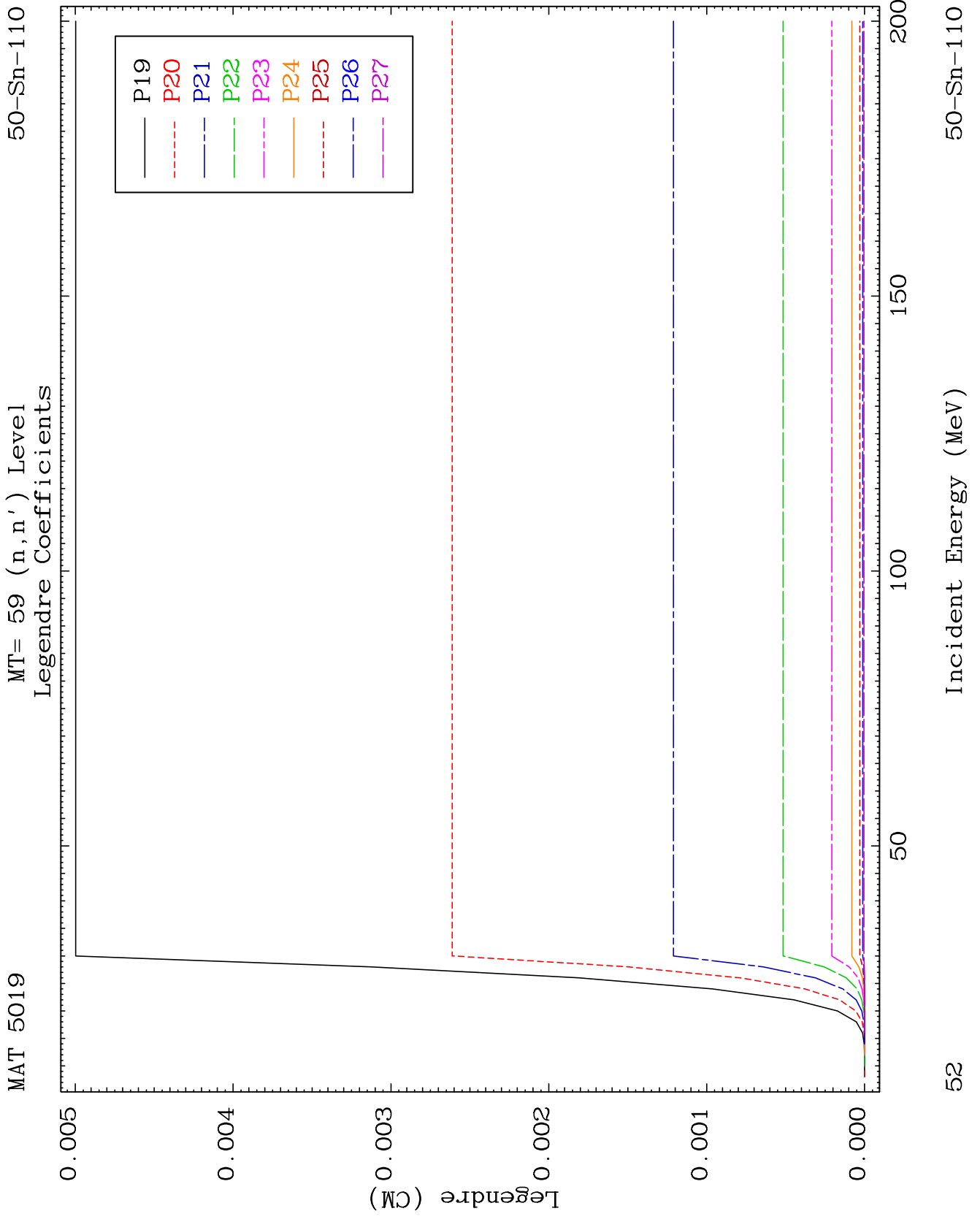
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Legendre Coefficients

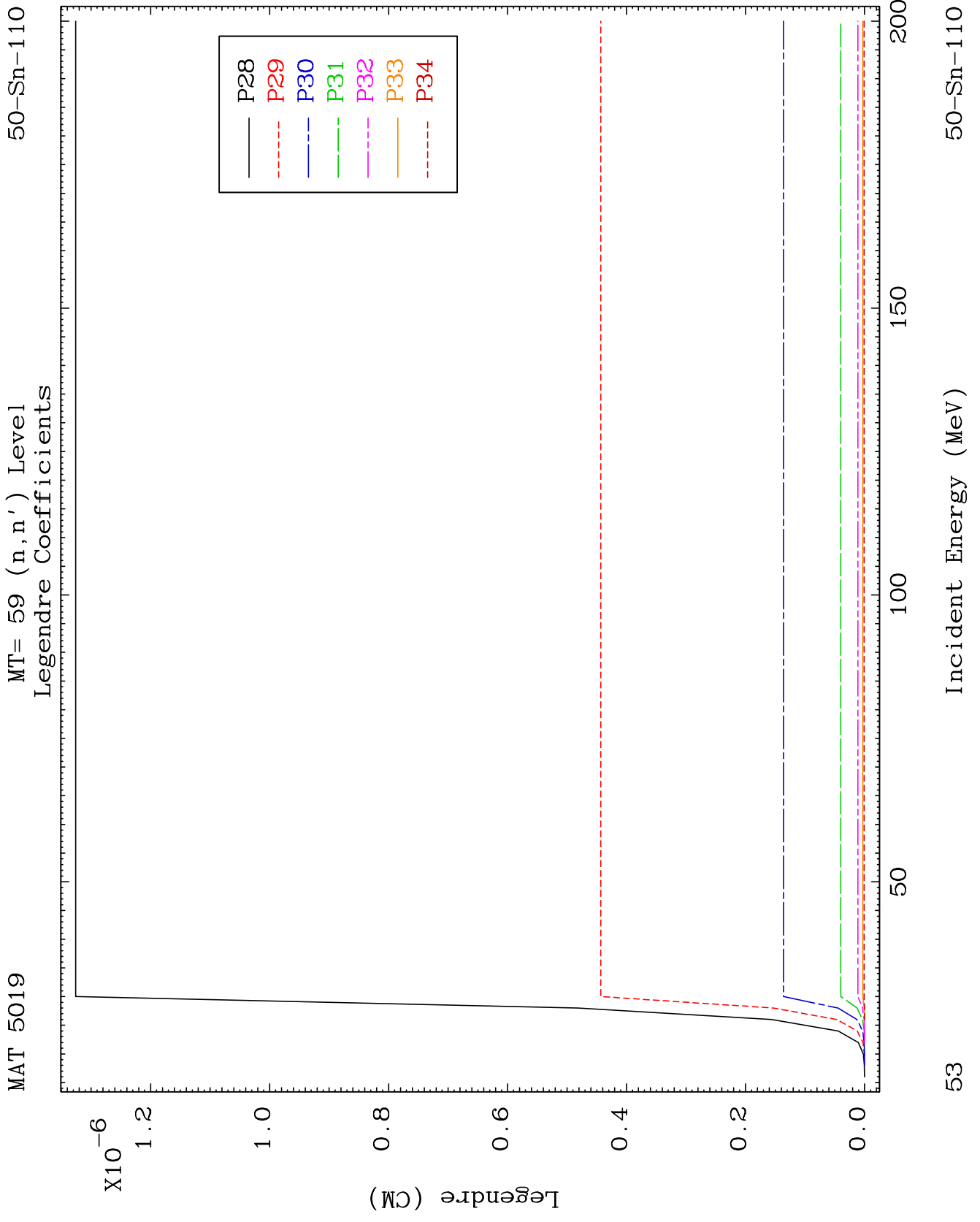
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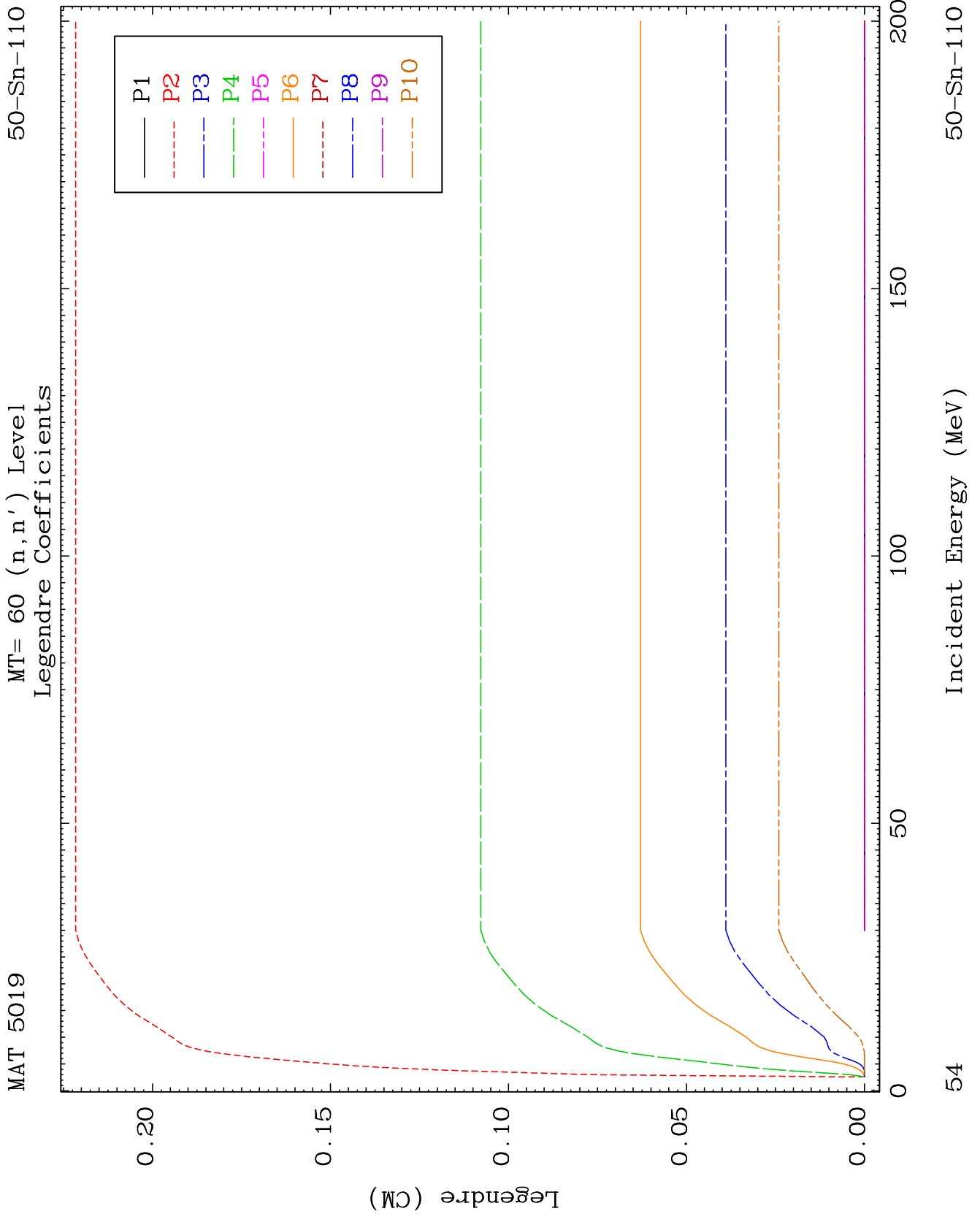


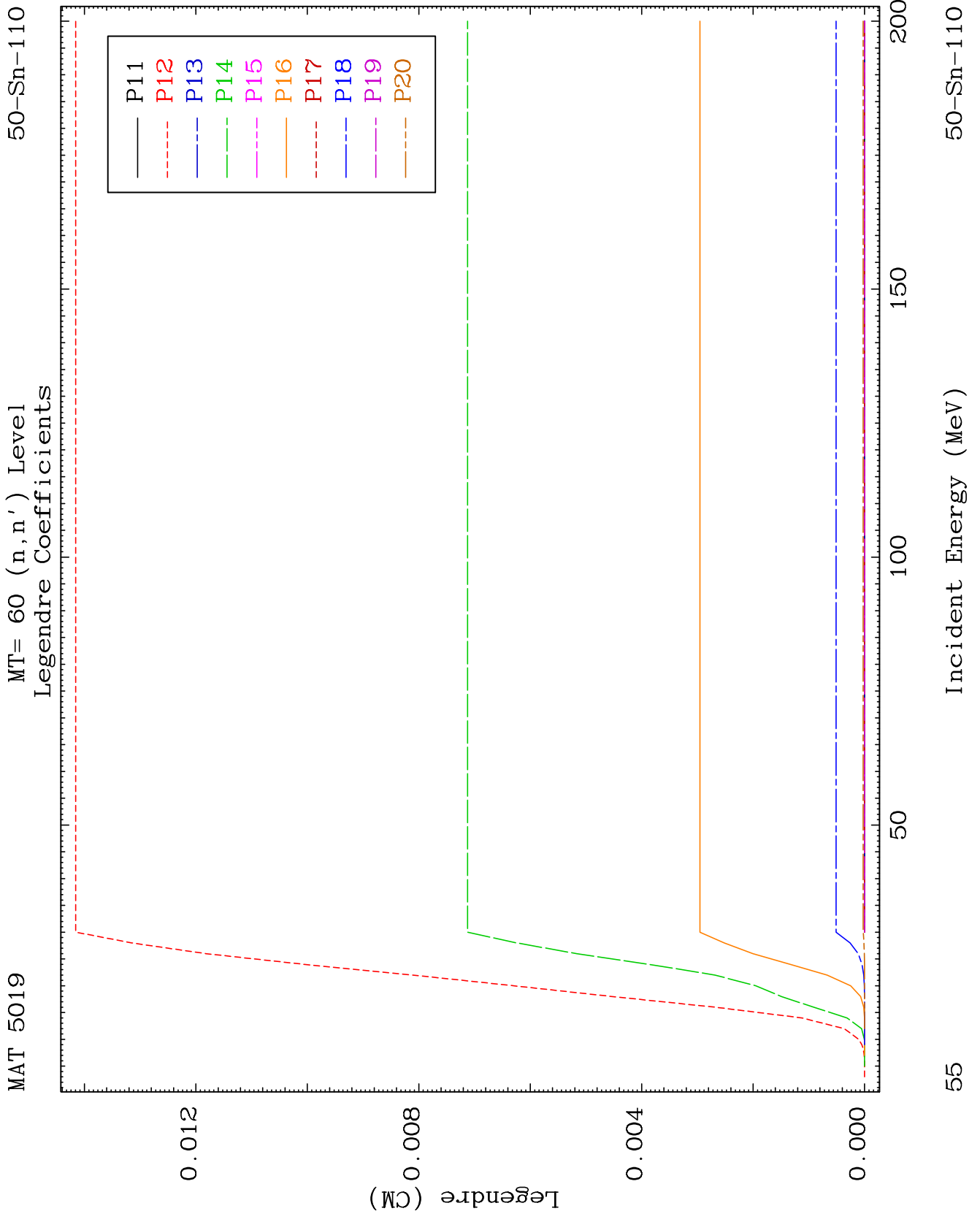


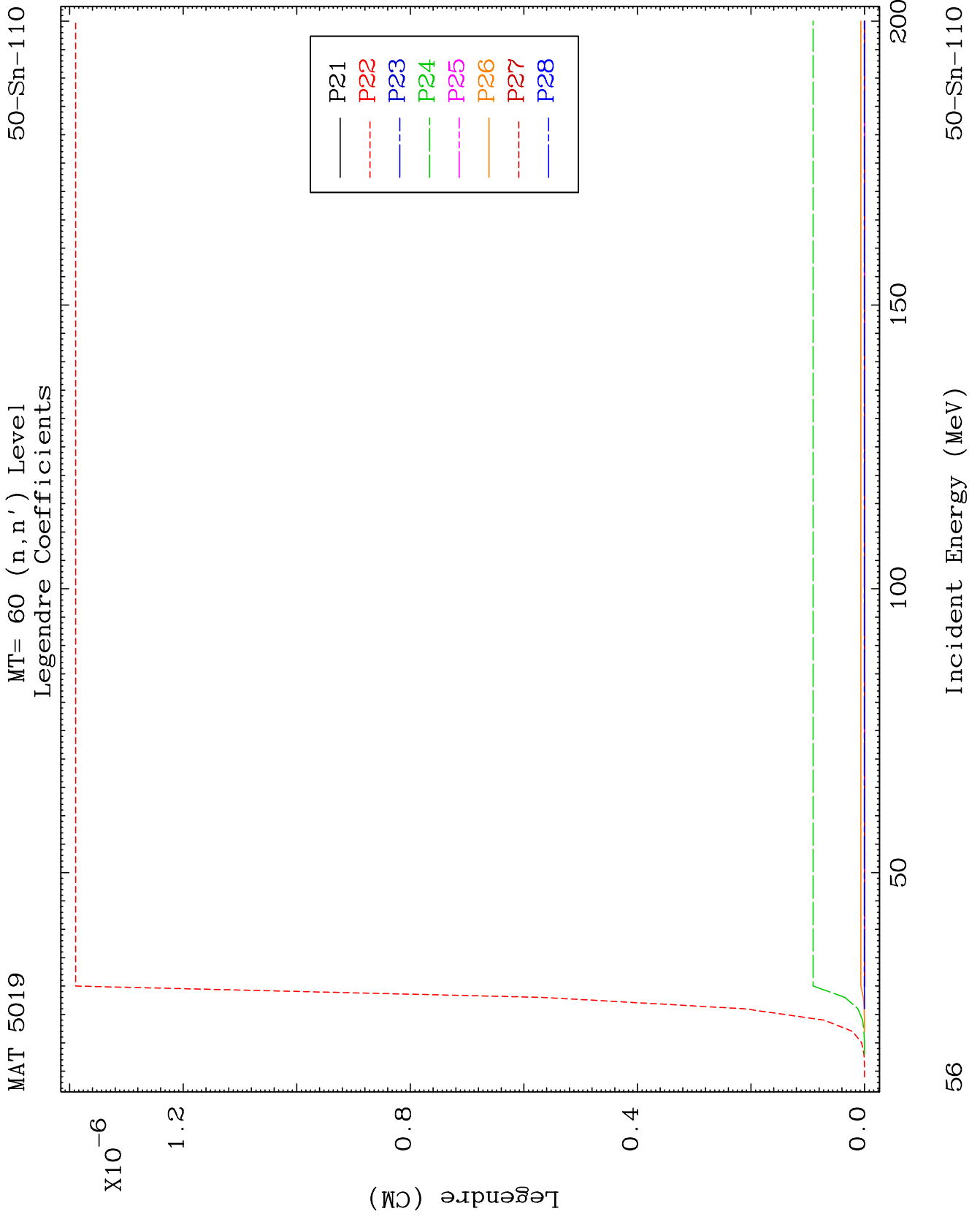


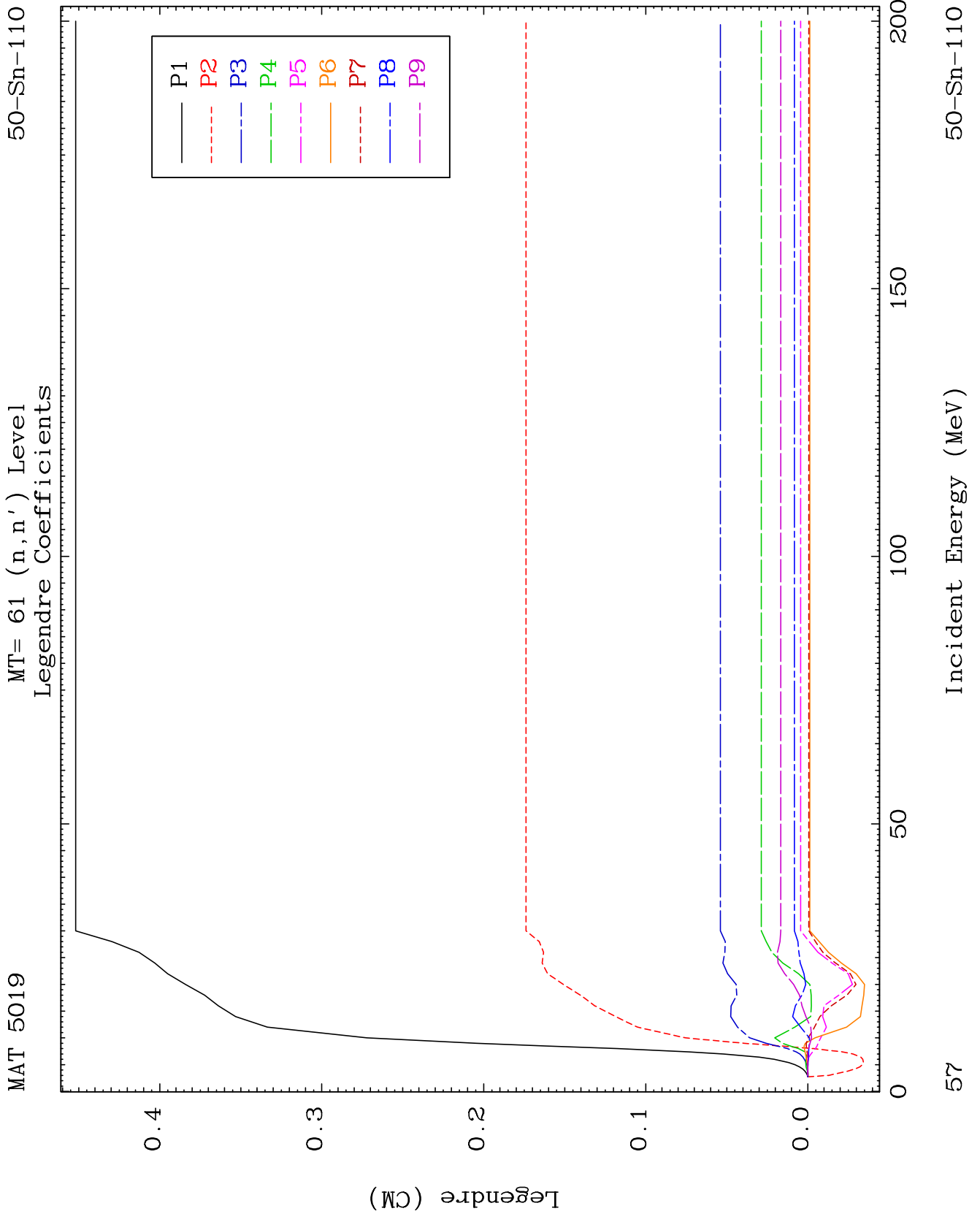


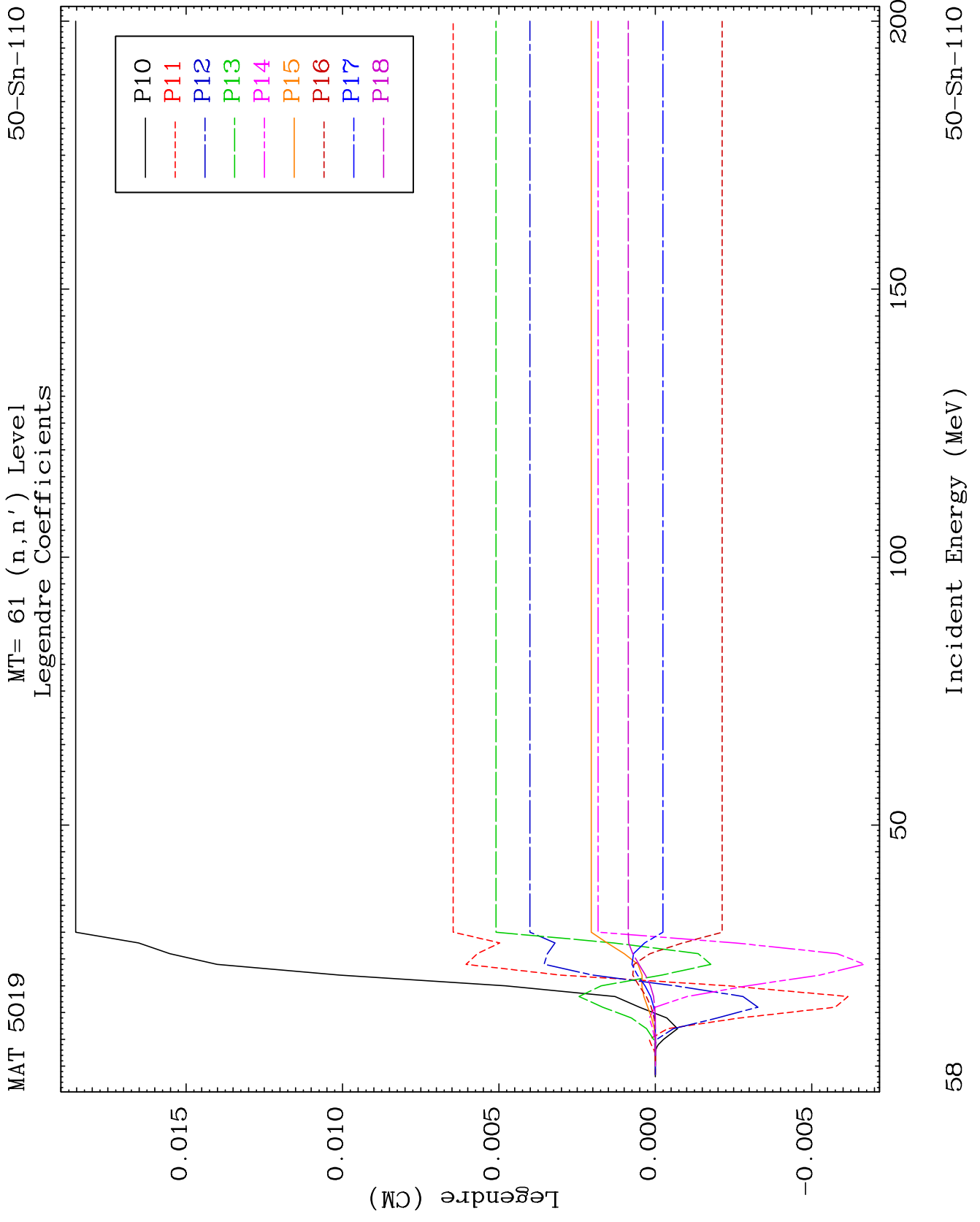








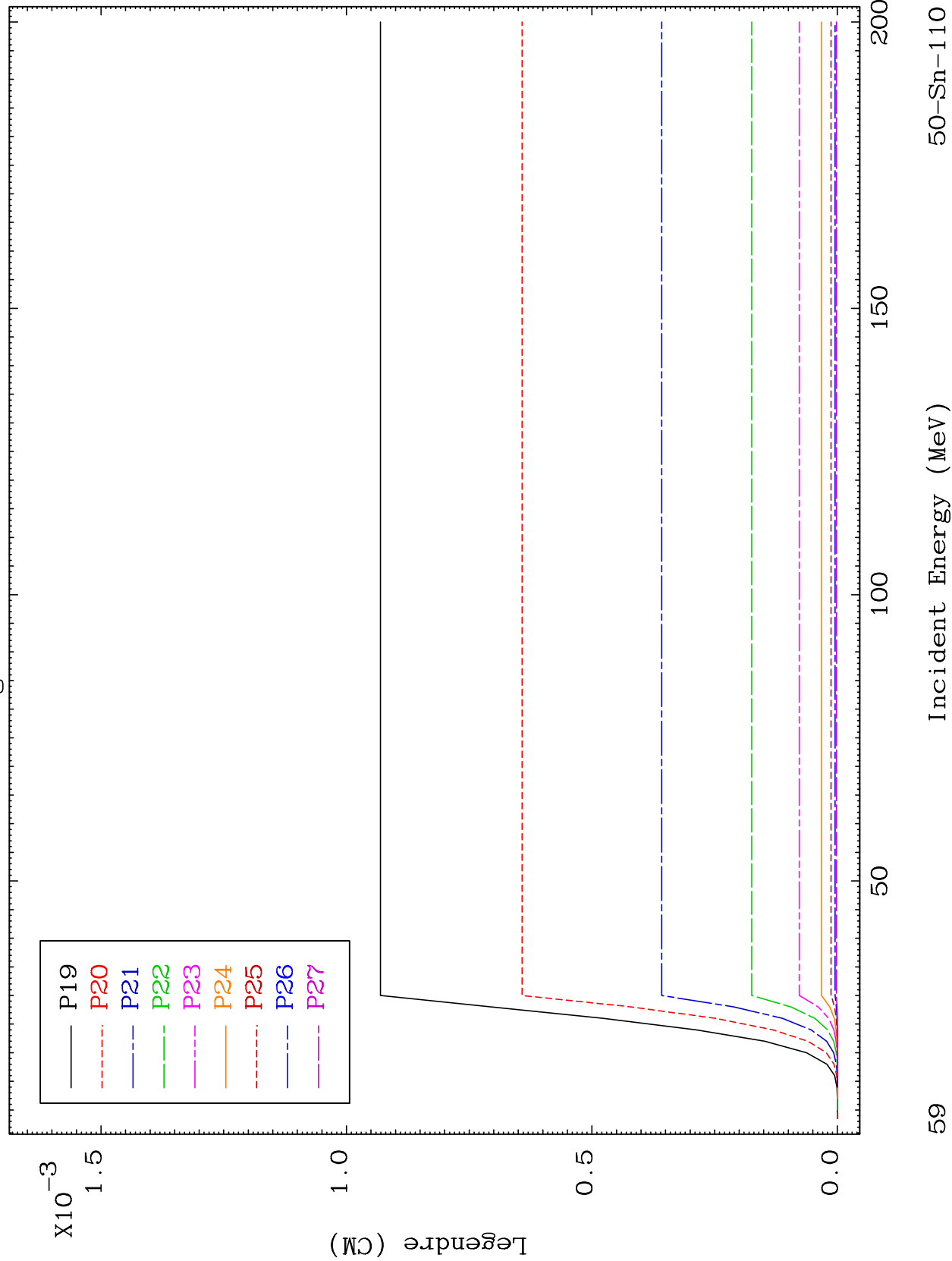
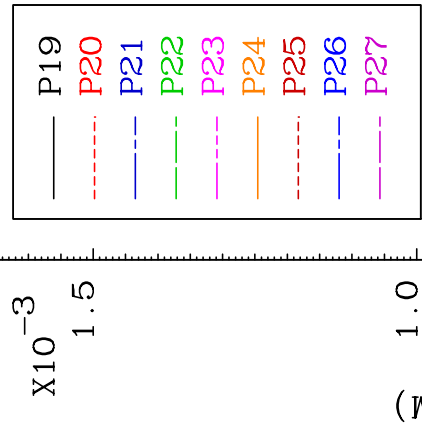




MAT 5019

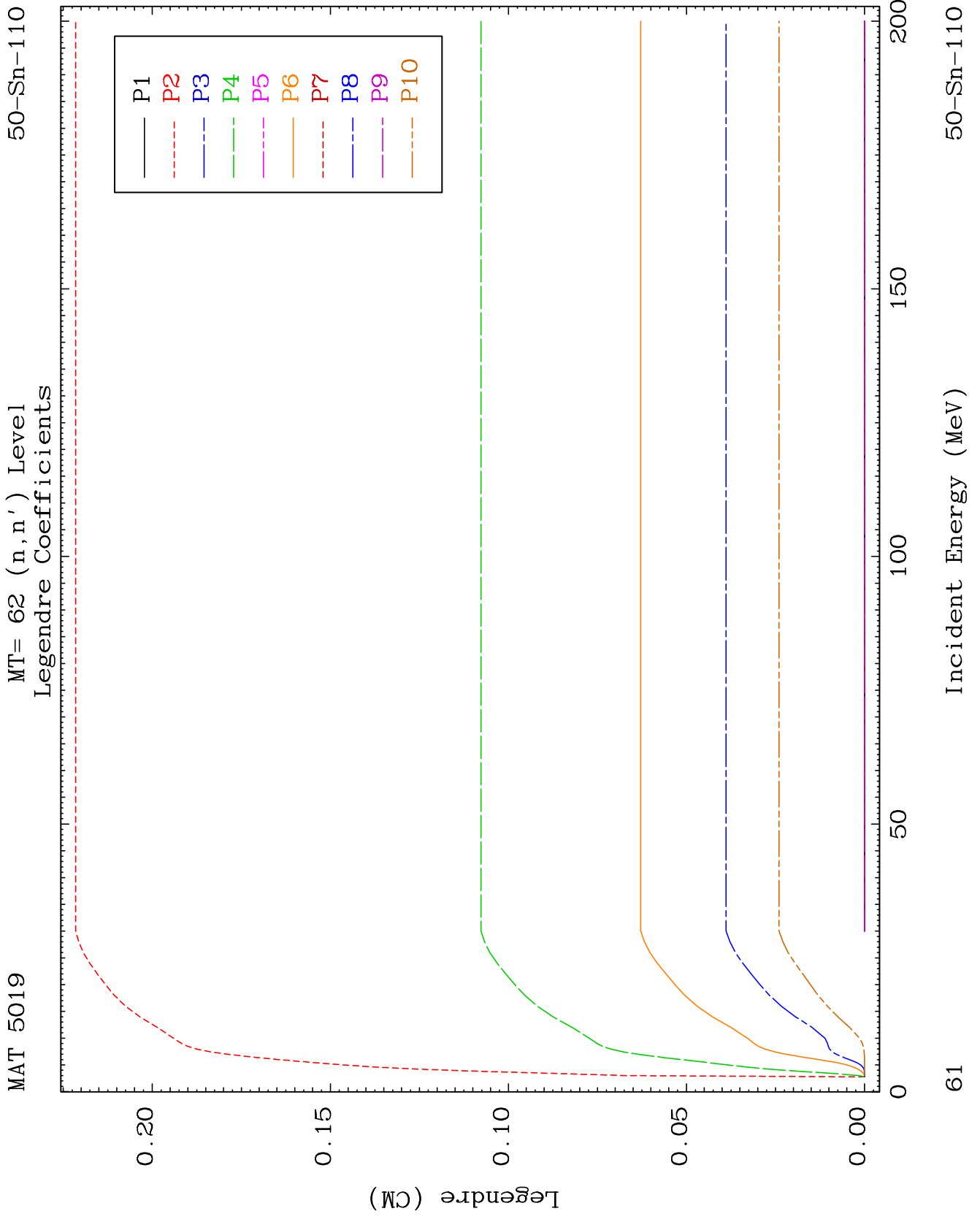
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Legendre Coefficients

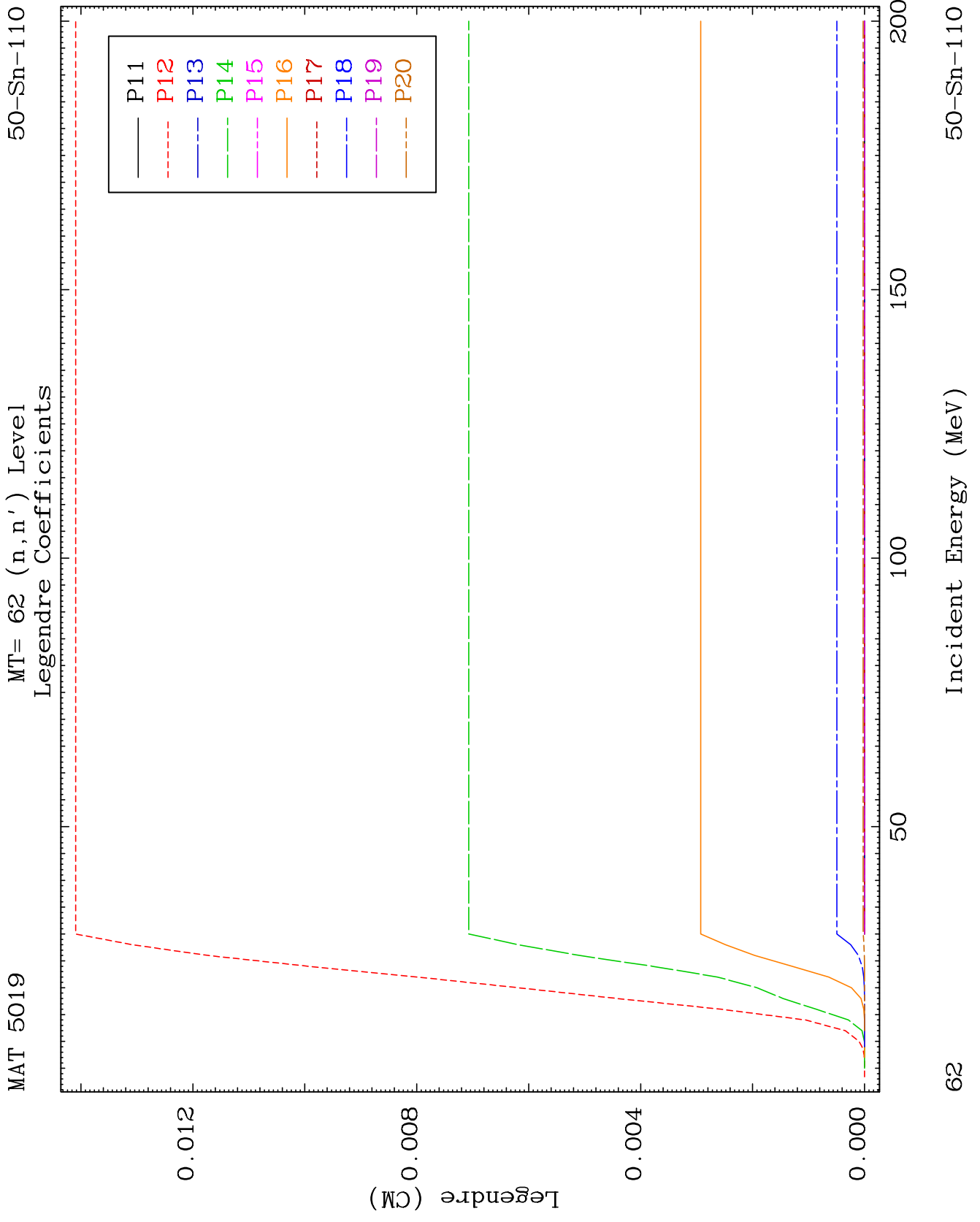
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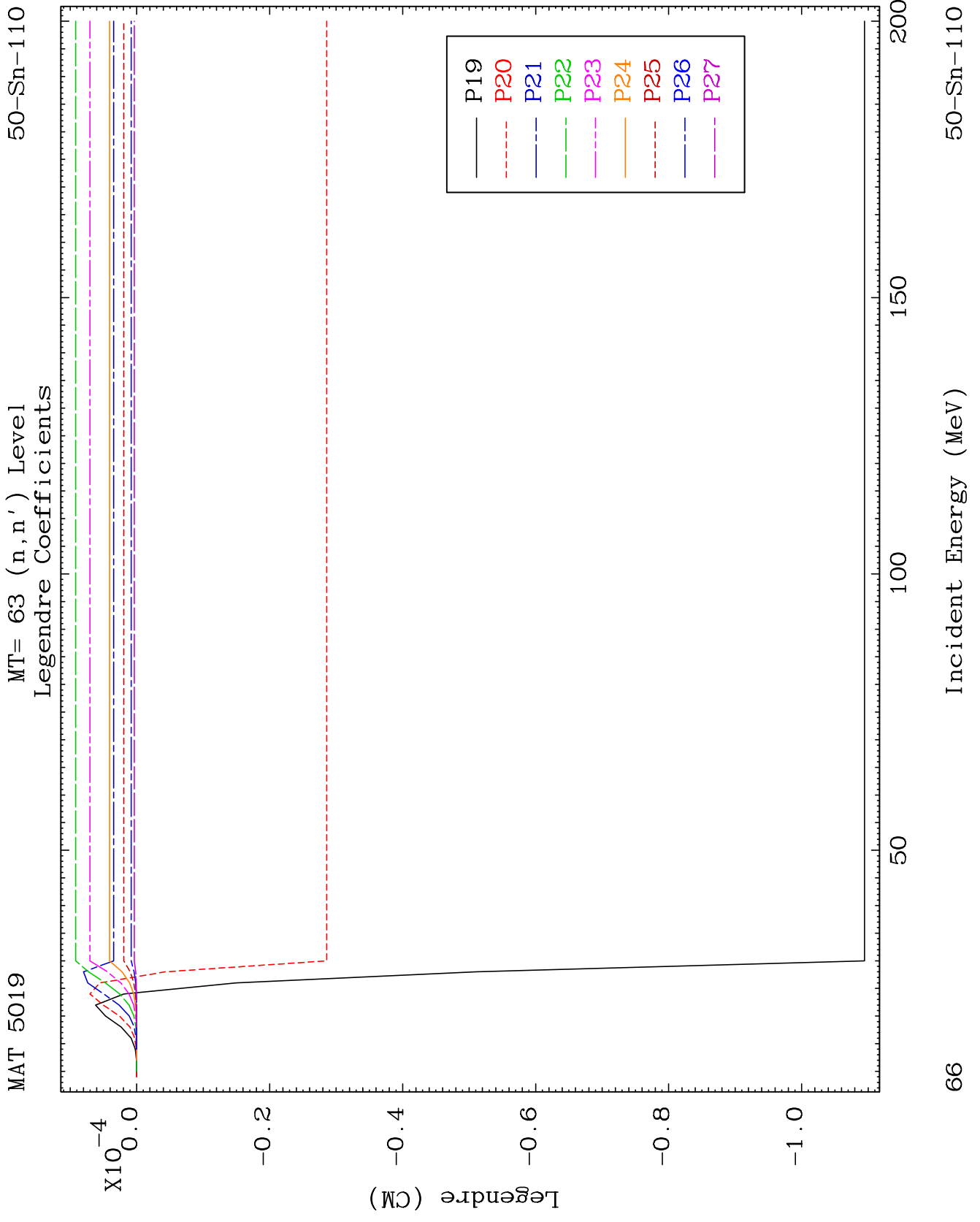


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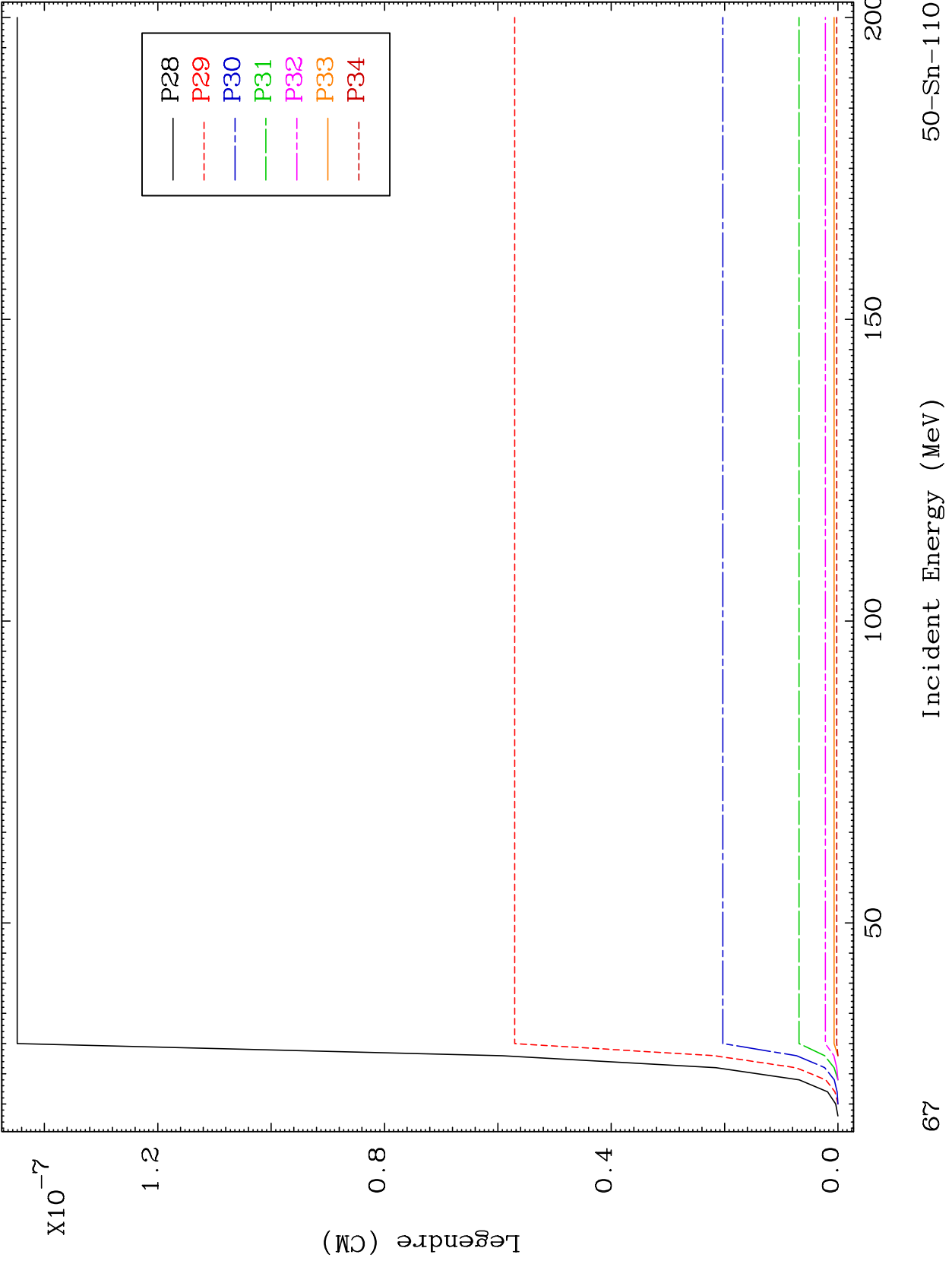
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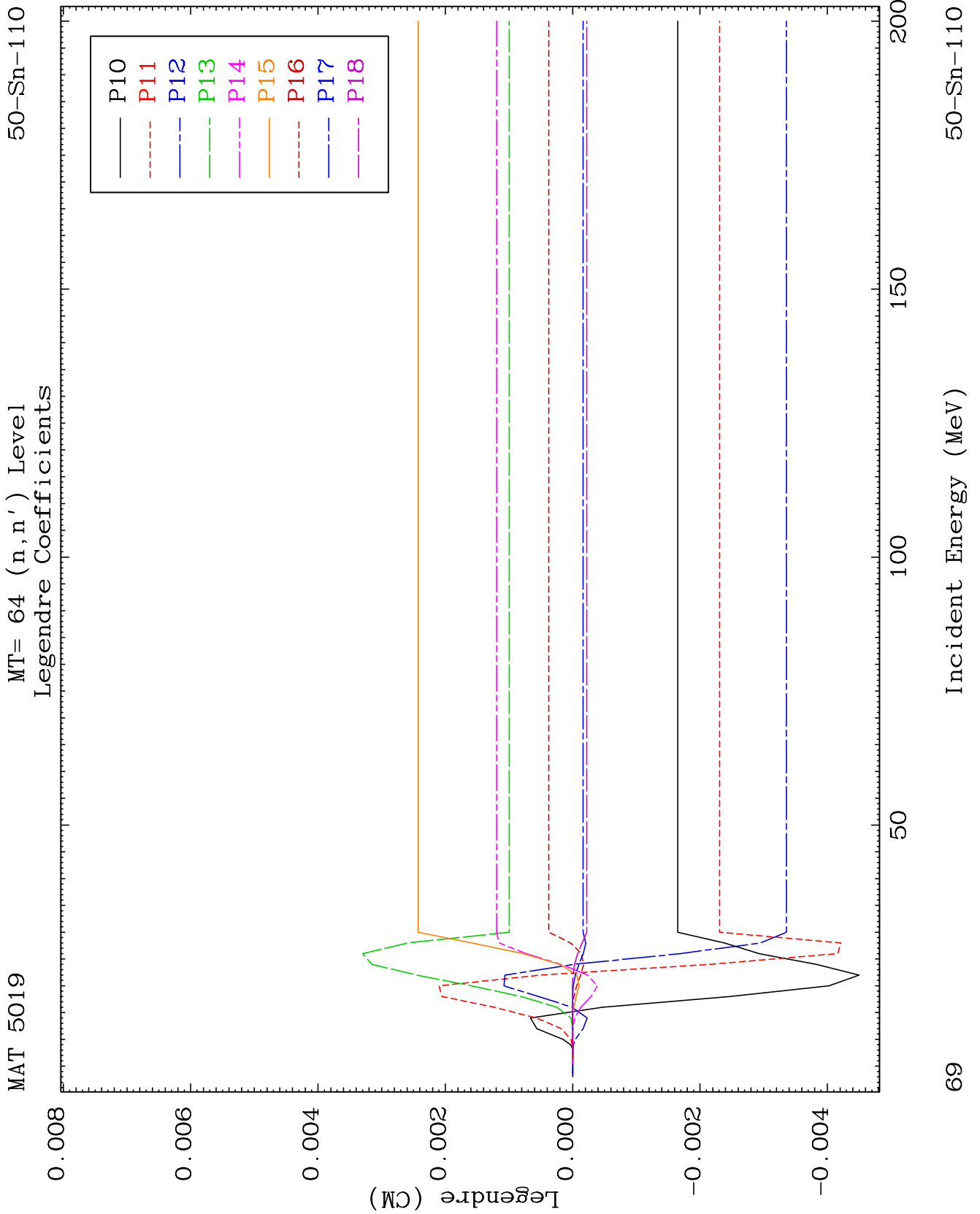


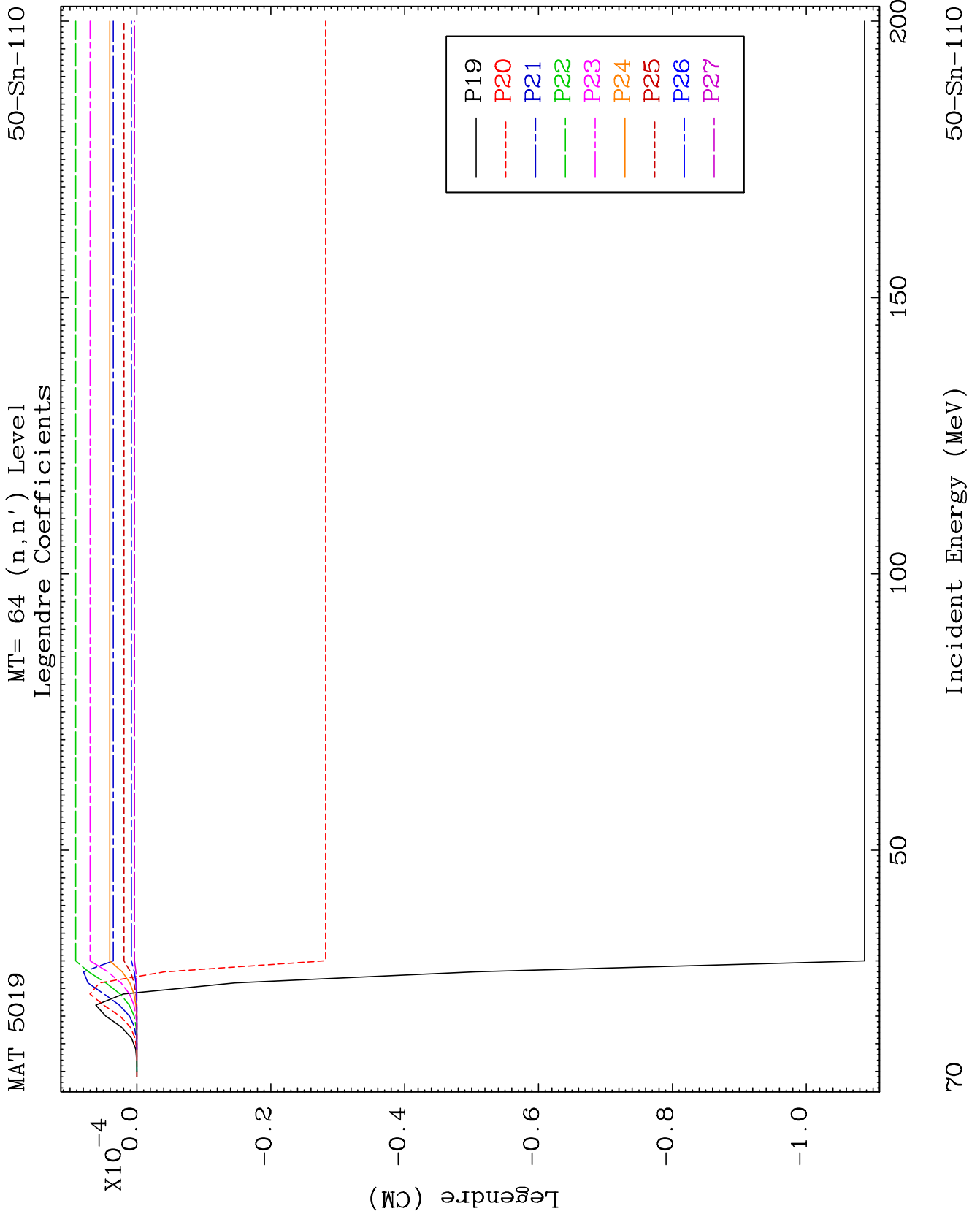




MAT 5019 MT= 63 (n,n') Level Legendre Coefficients 50-Sn-110



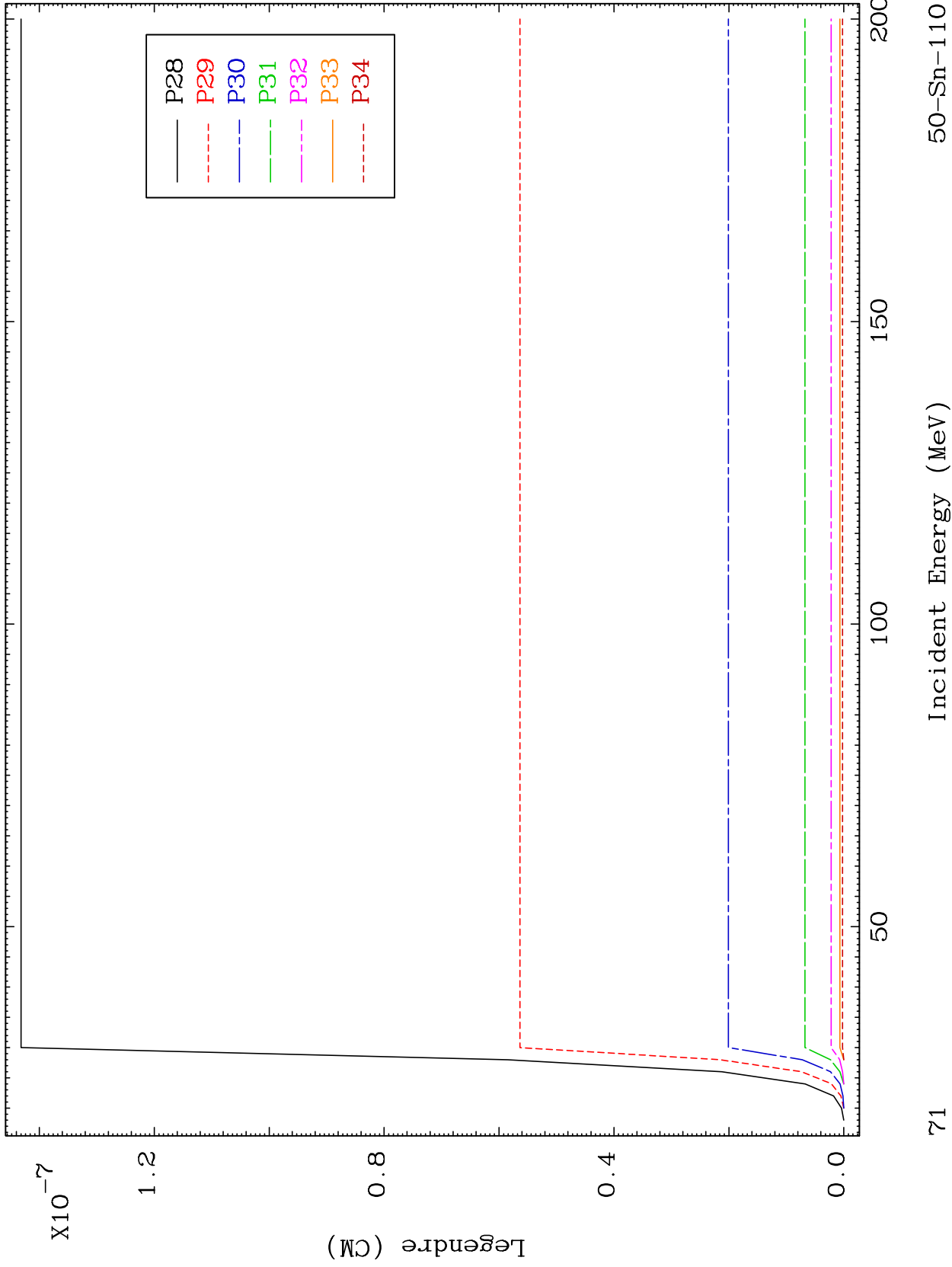


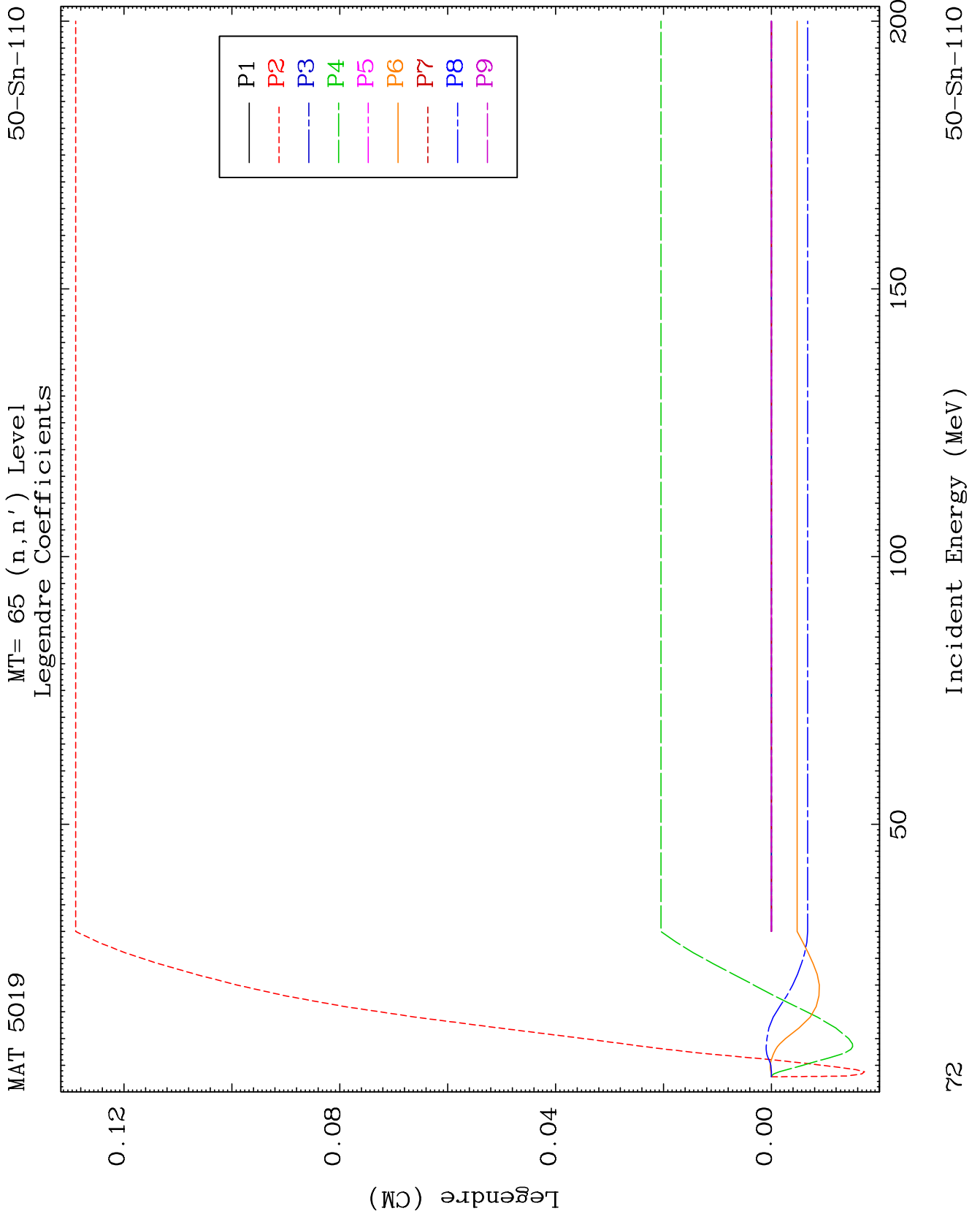


MAT 5019

MT= 64 (n,n') Level
Legendre Coefficients

50-Sn-110

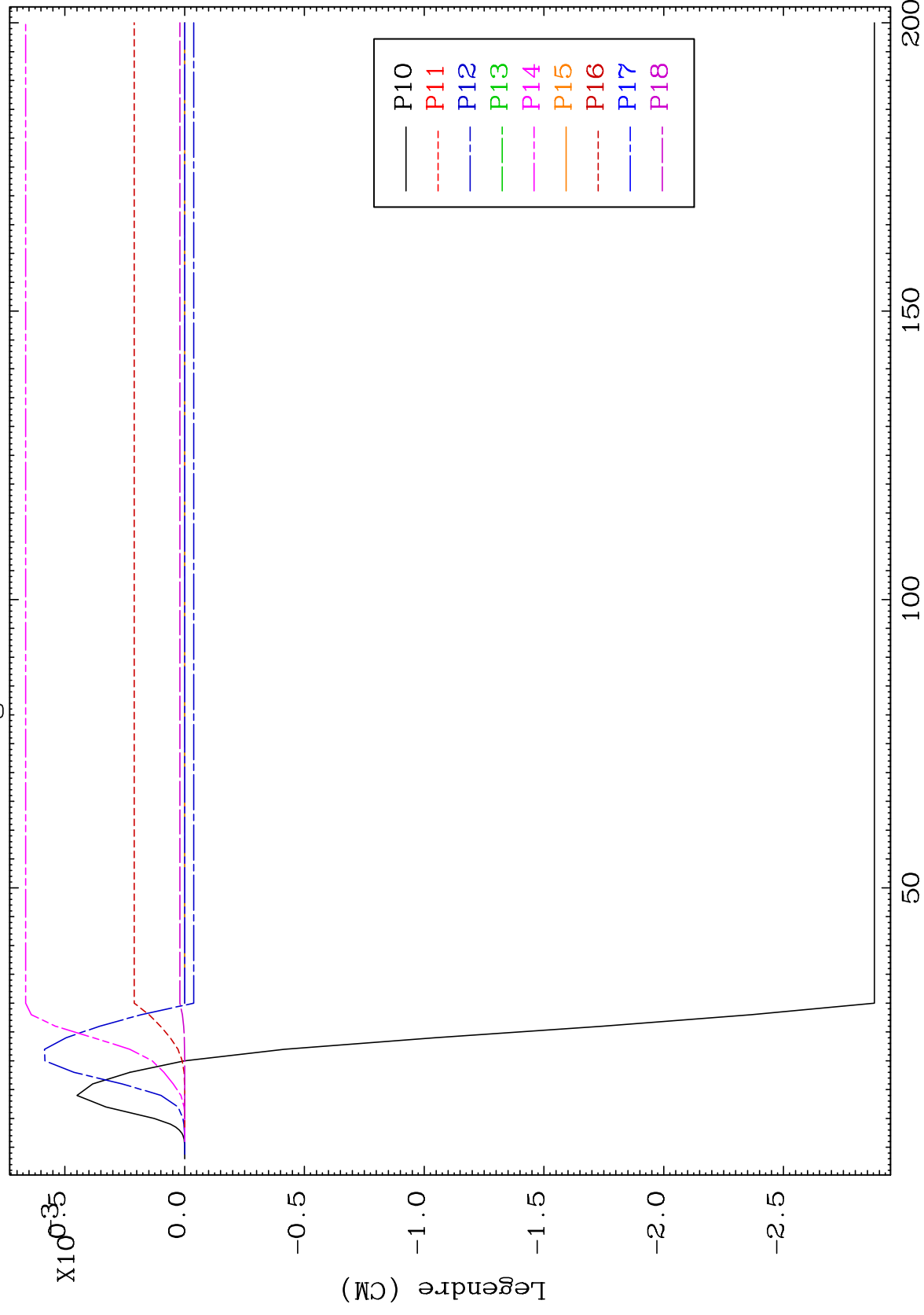


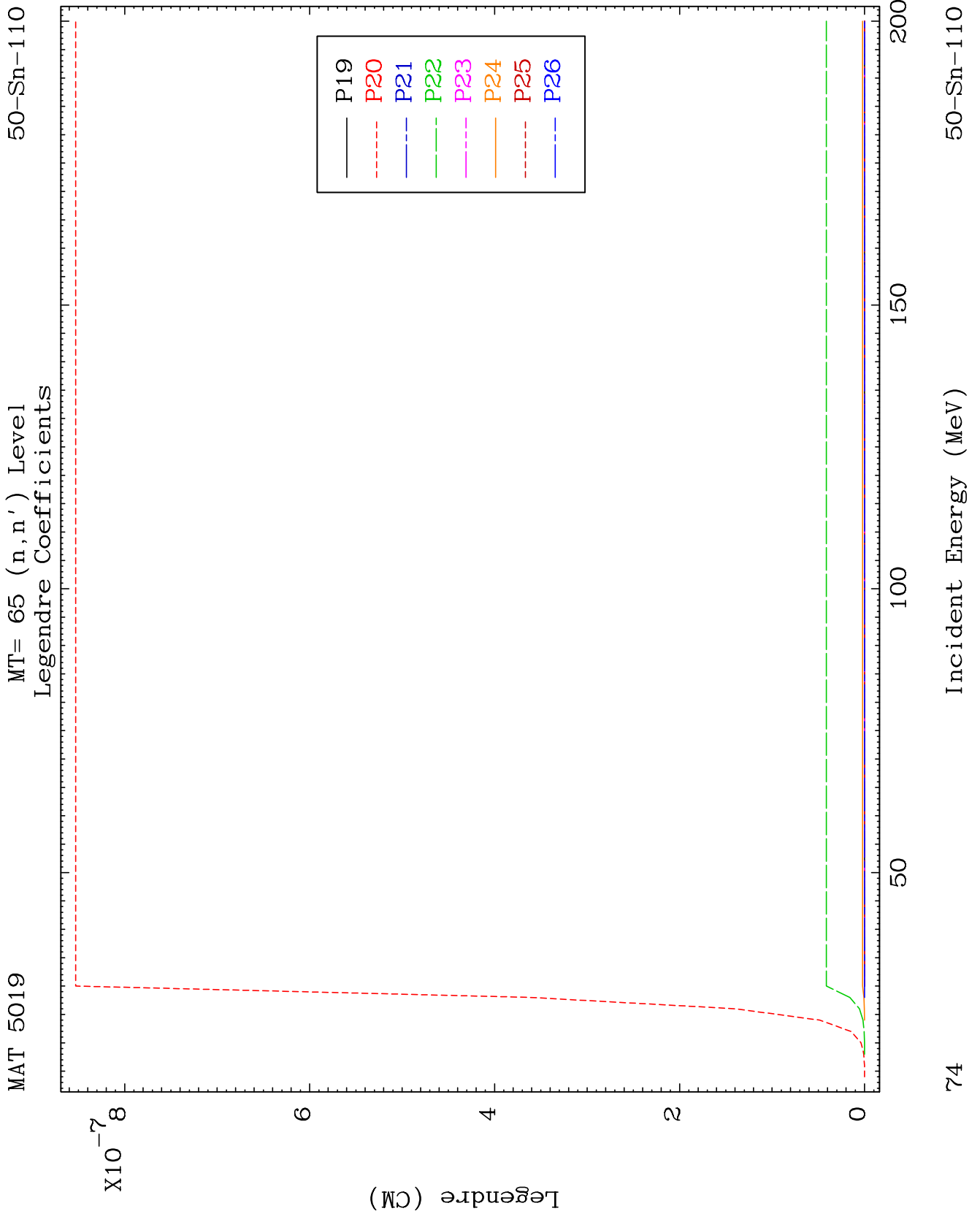


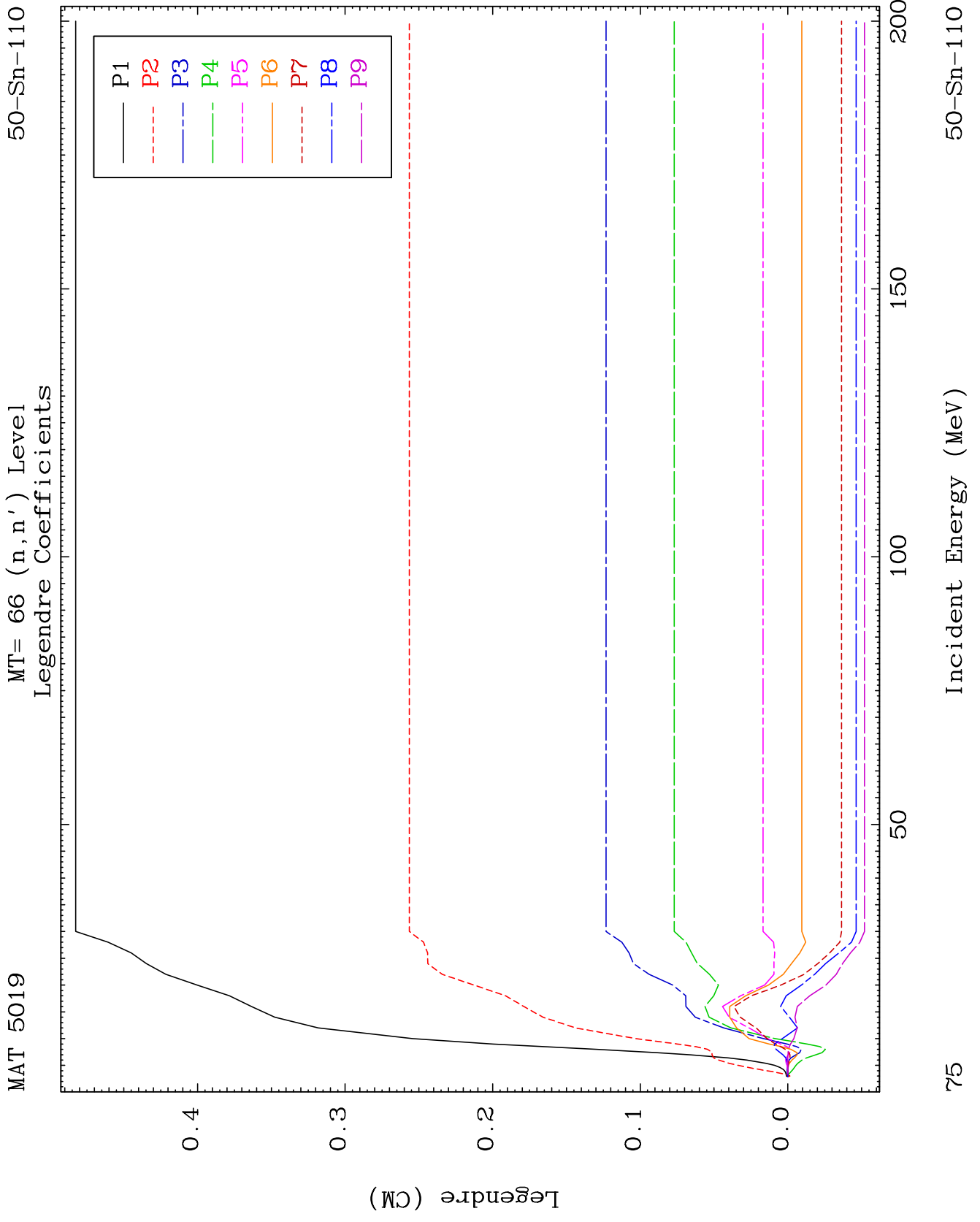
MAT 5019

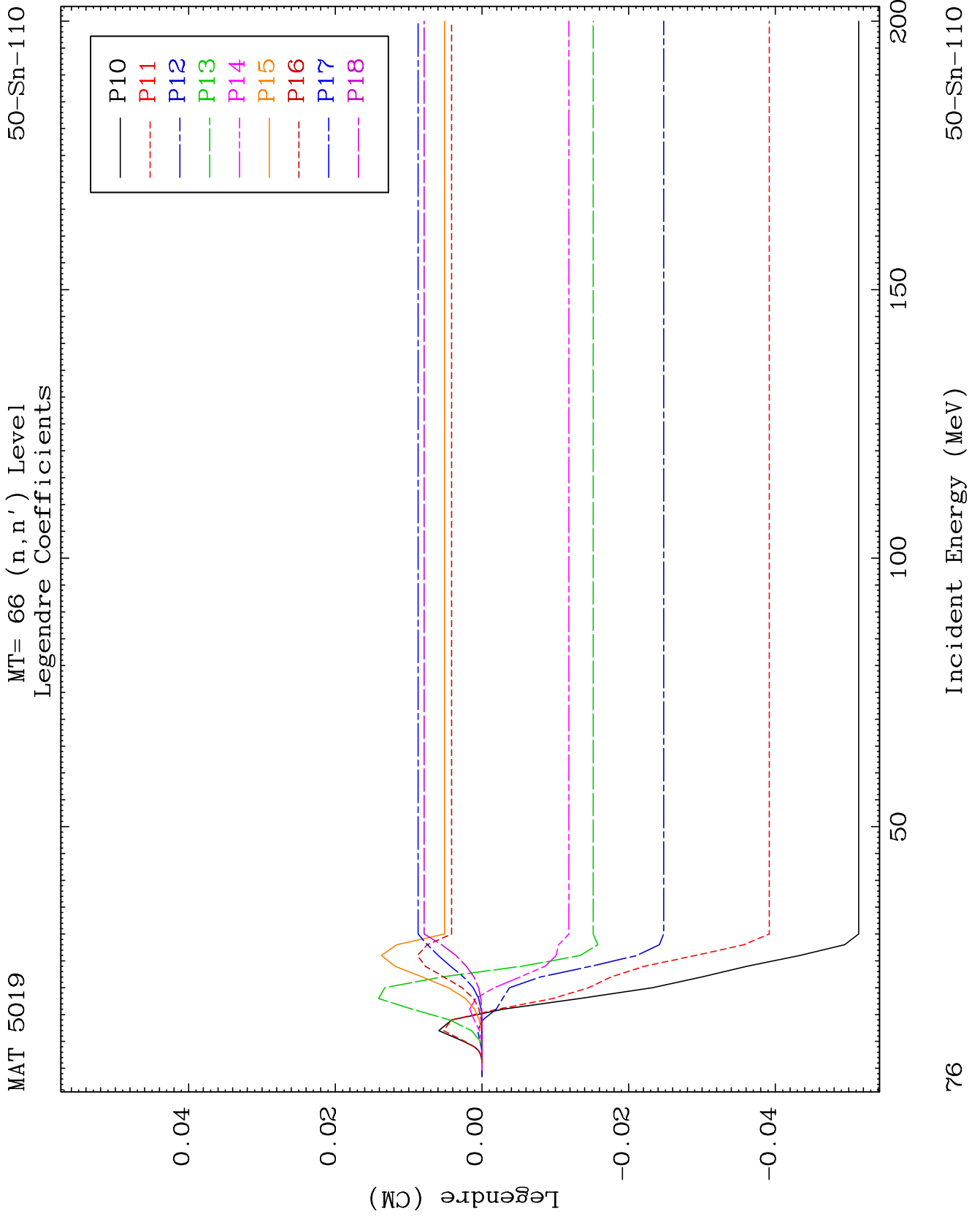
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Legendre Coefficients

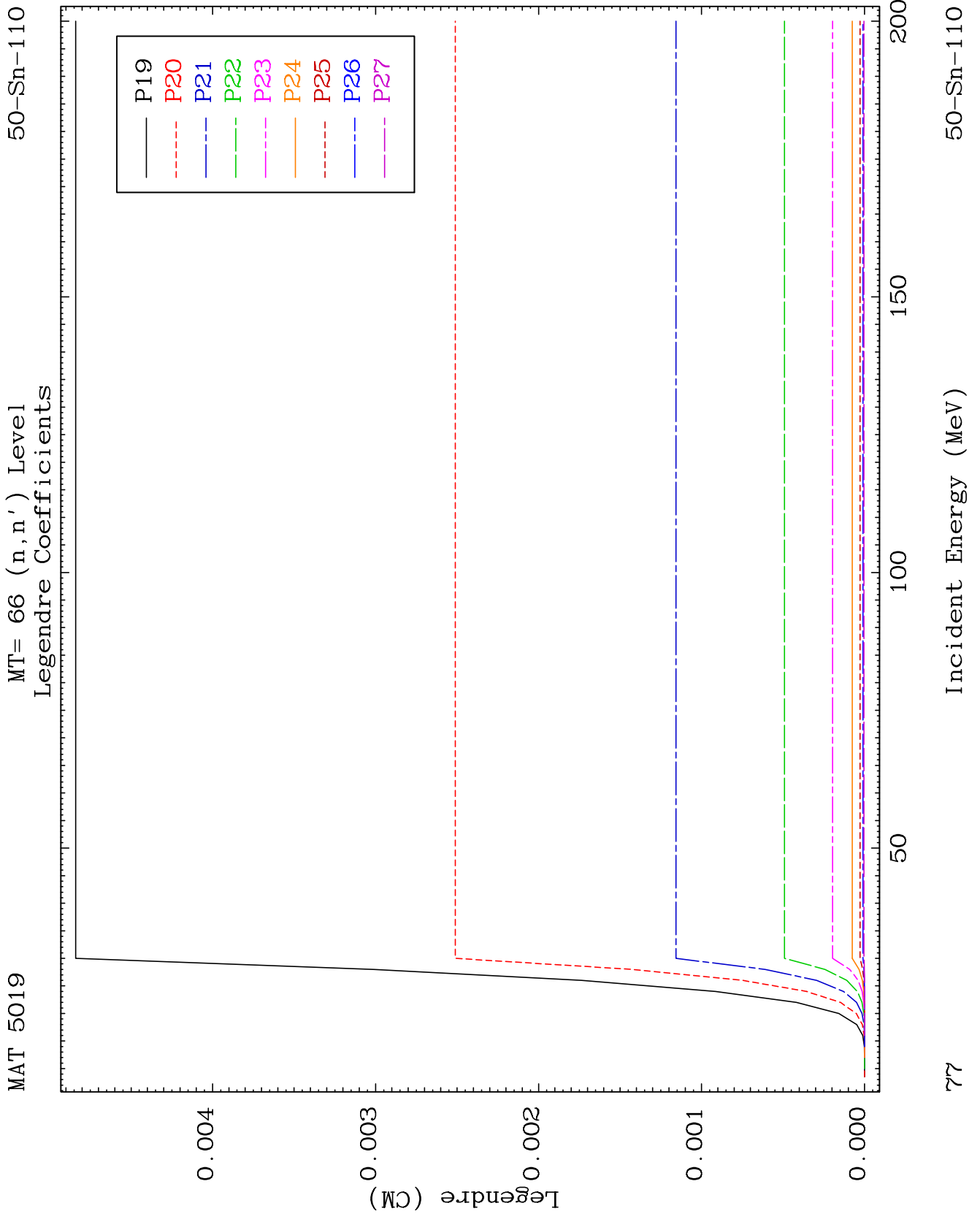
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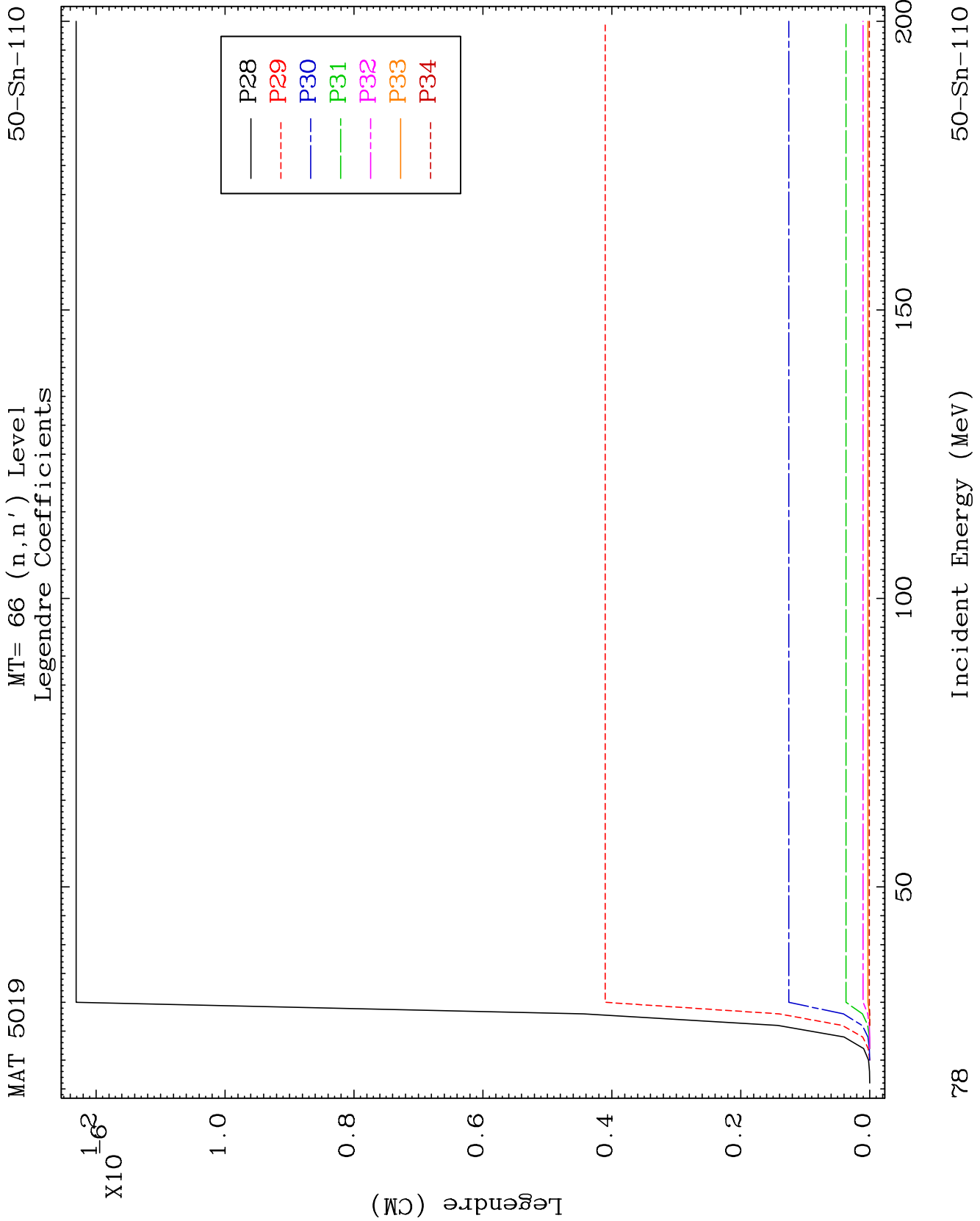


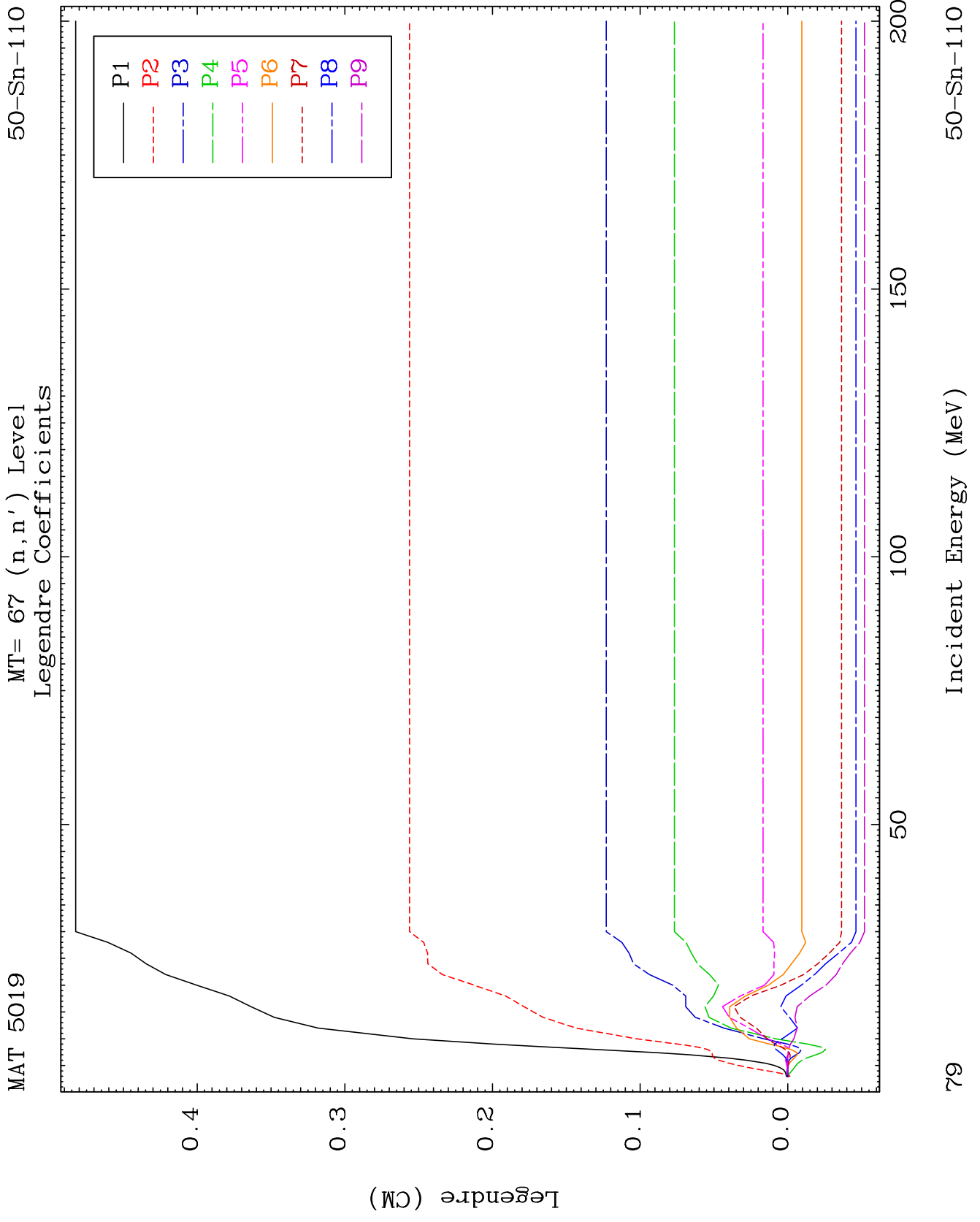


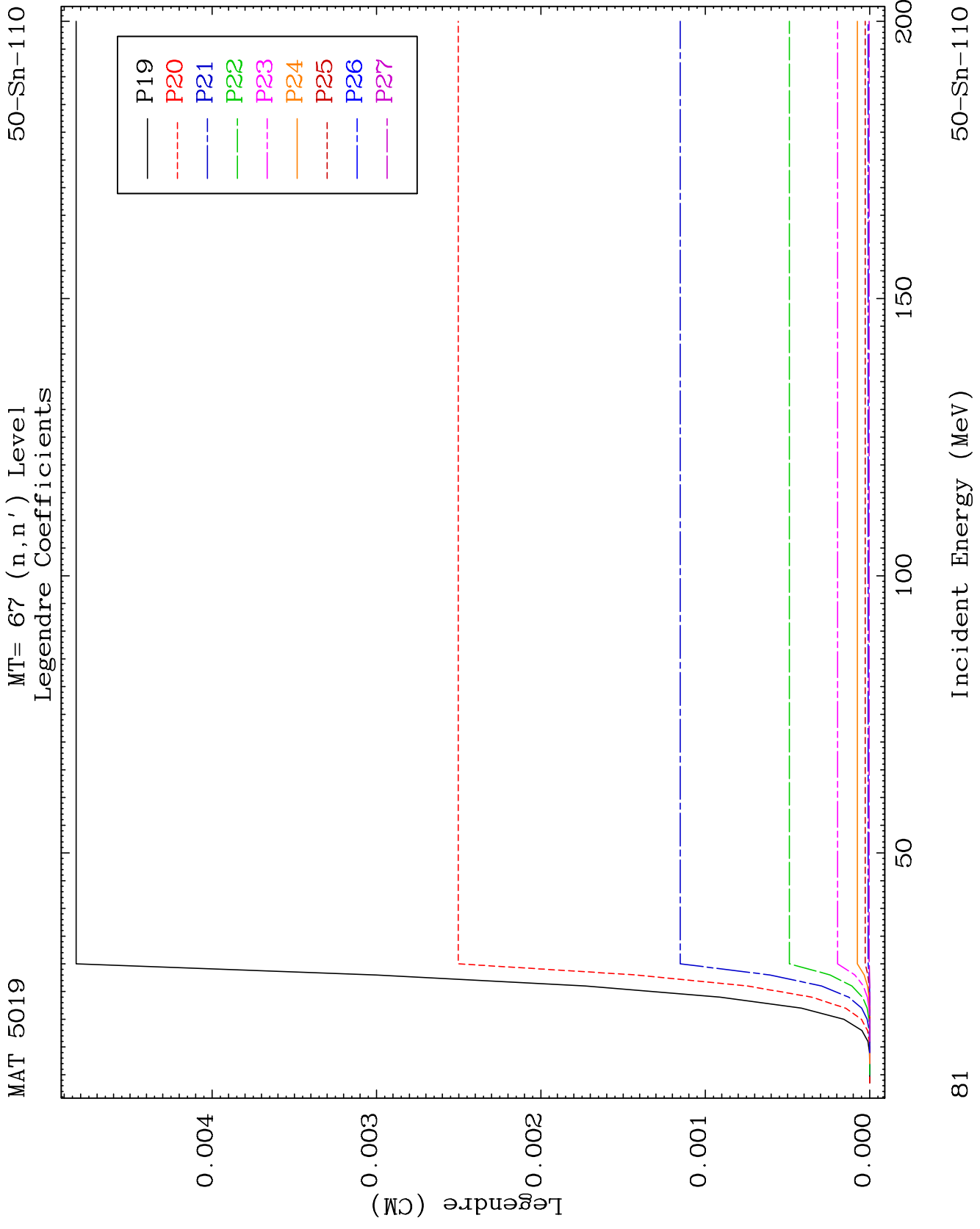








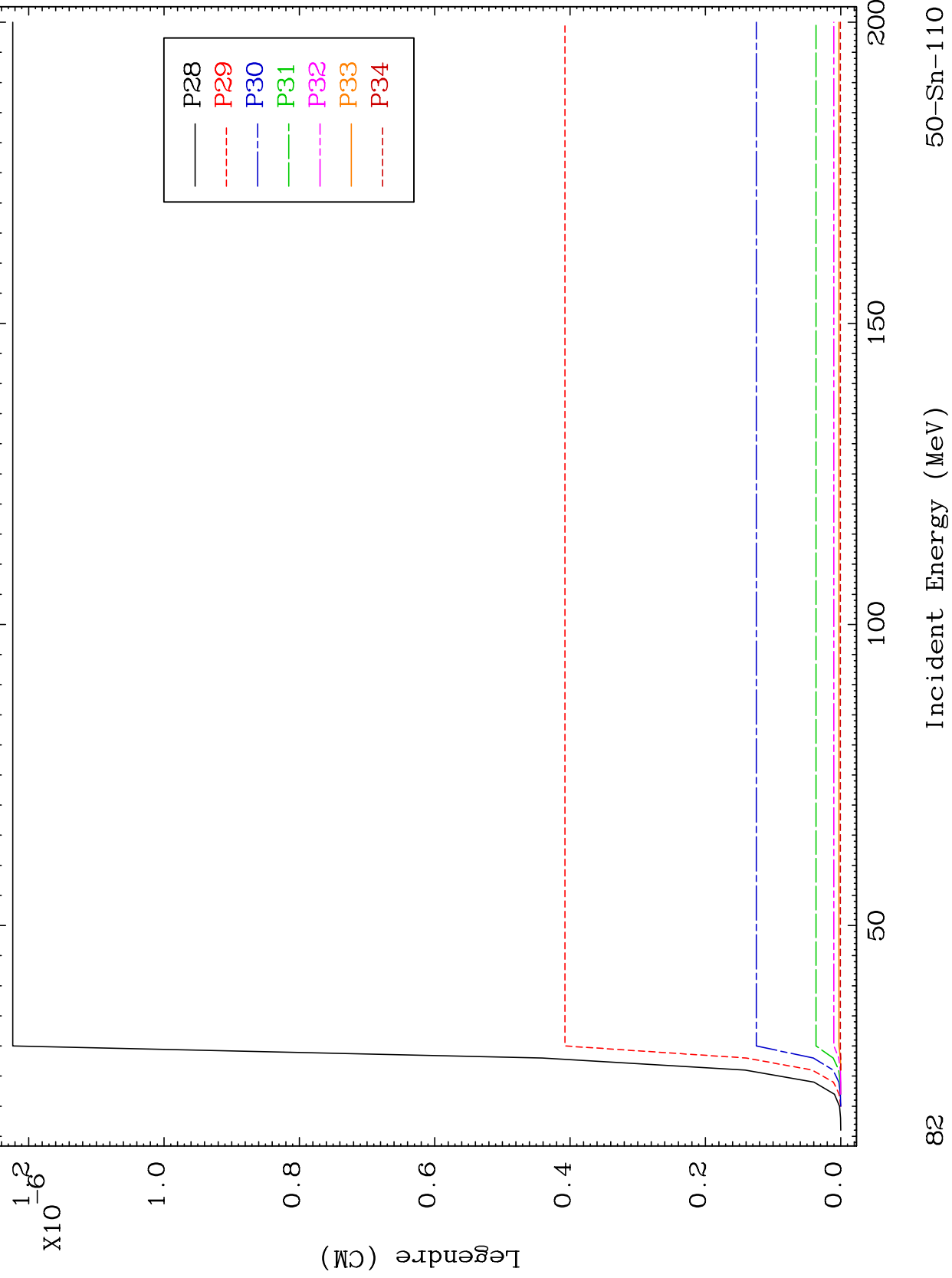




MAT 5019

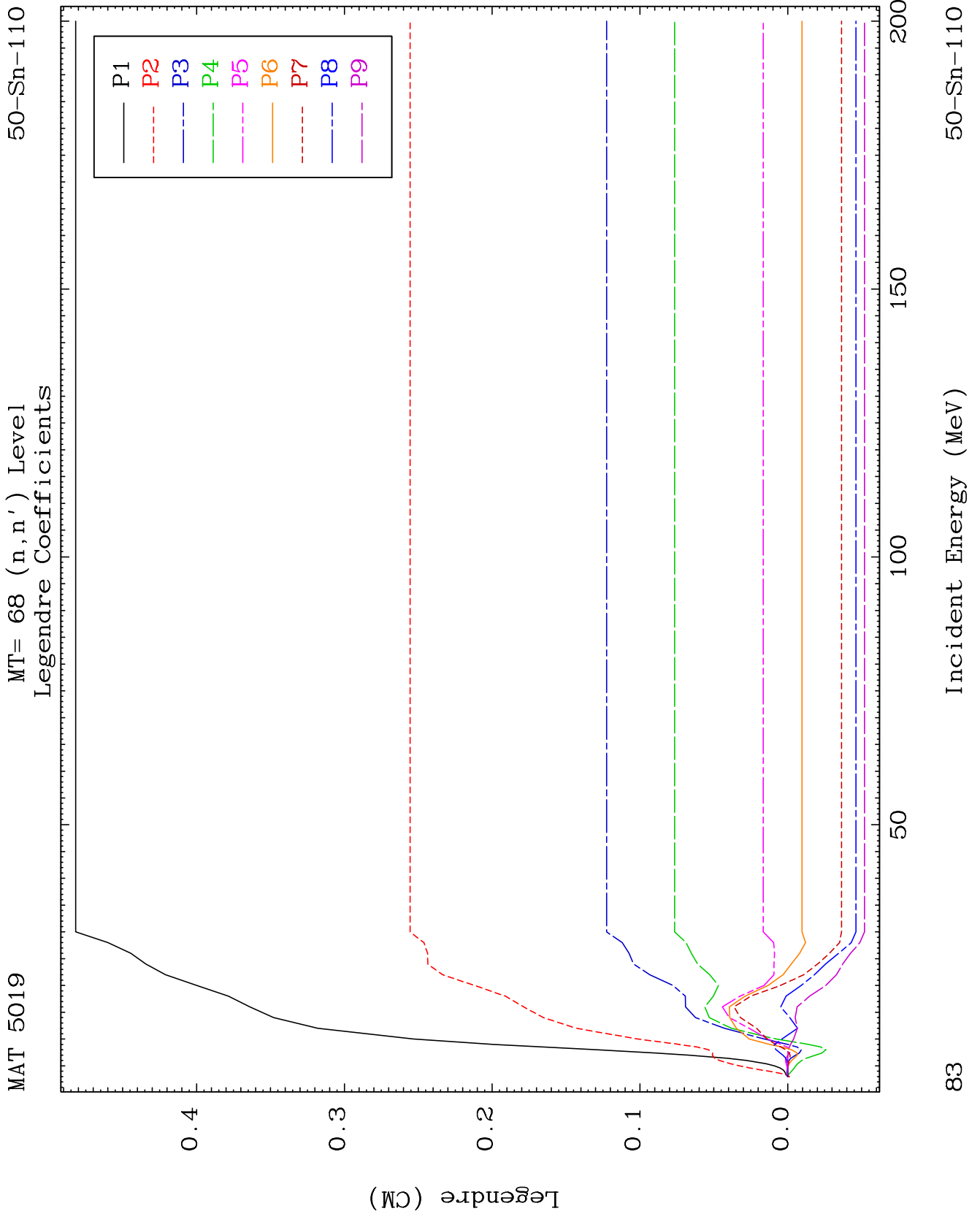
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Legendre Coefficients

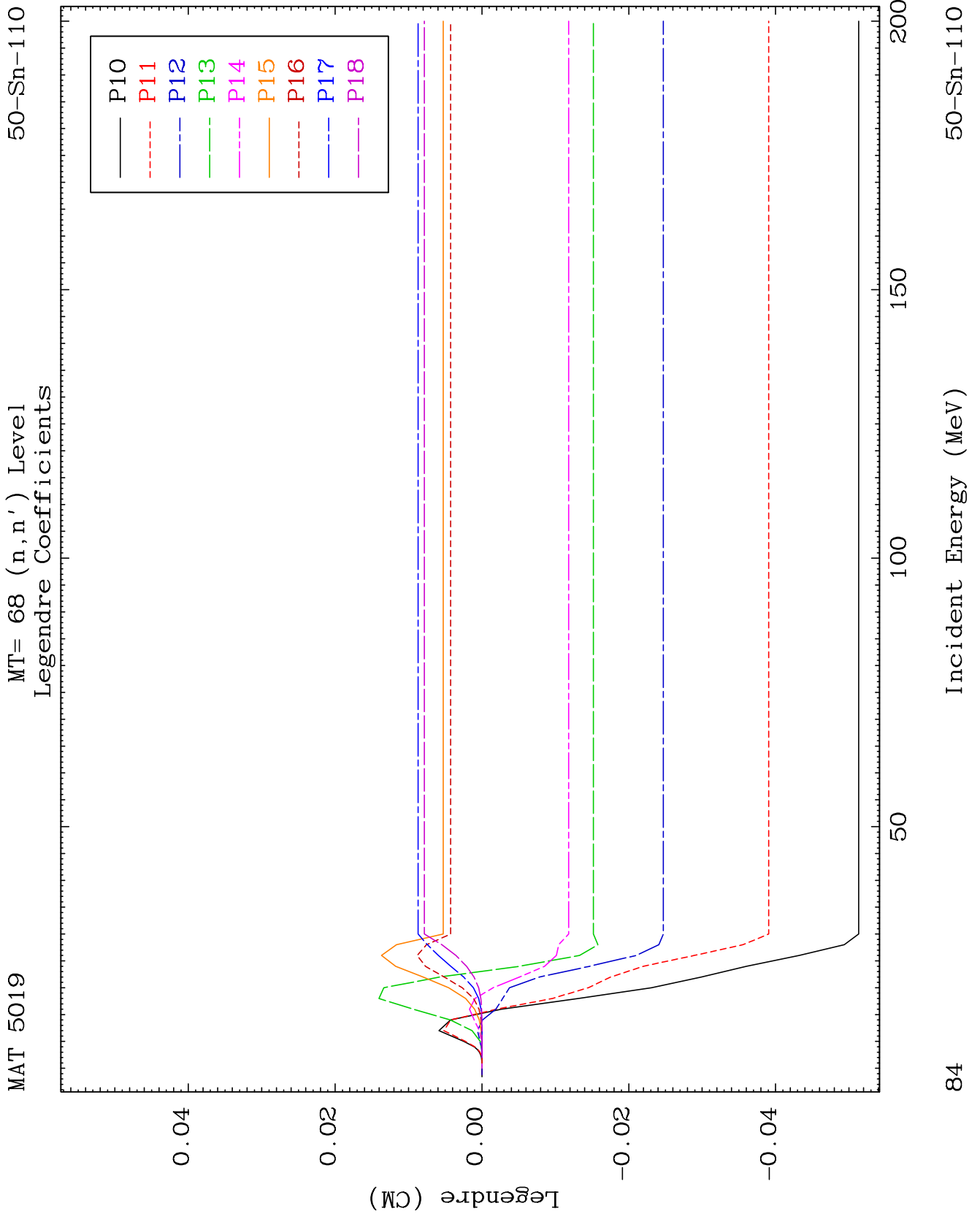
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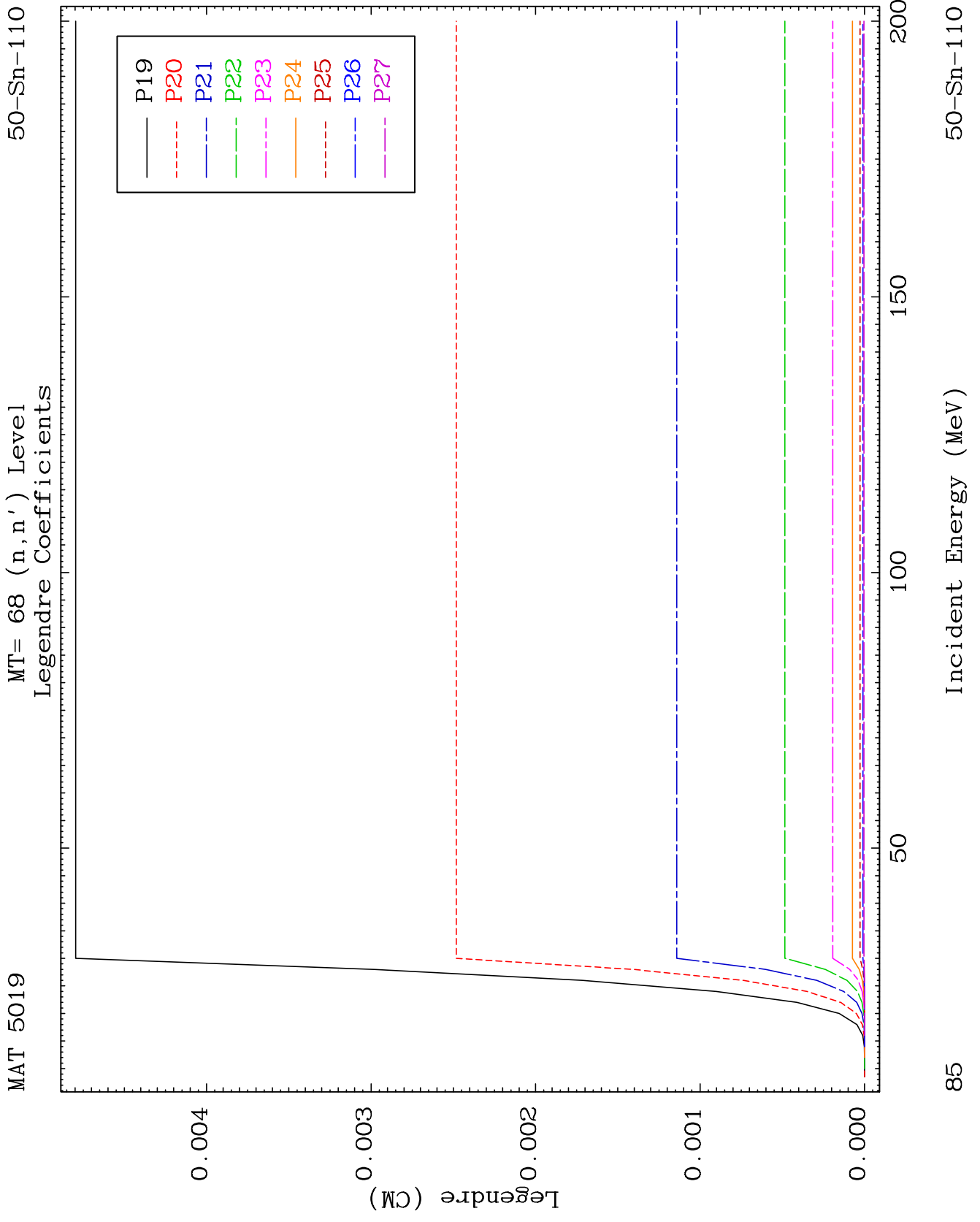


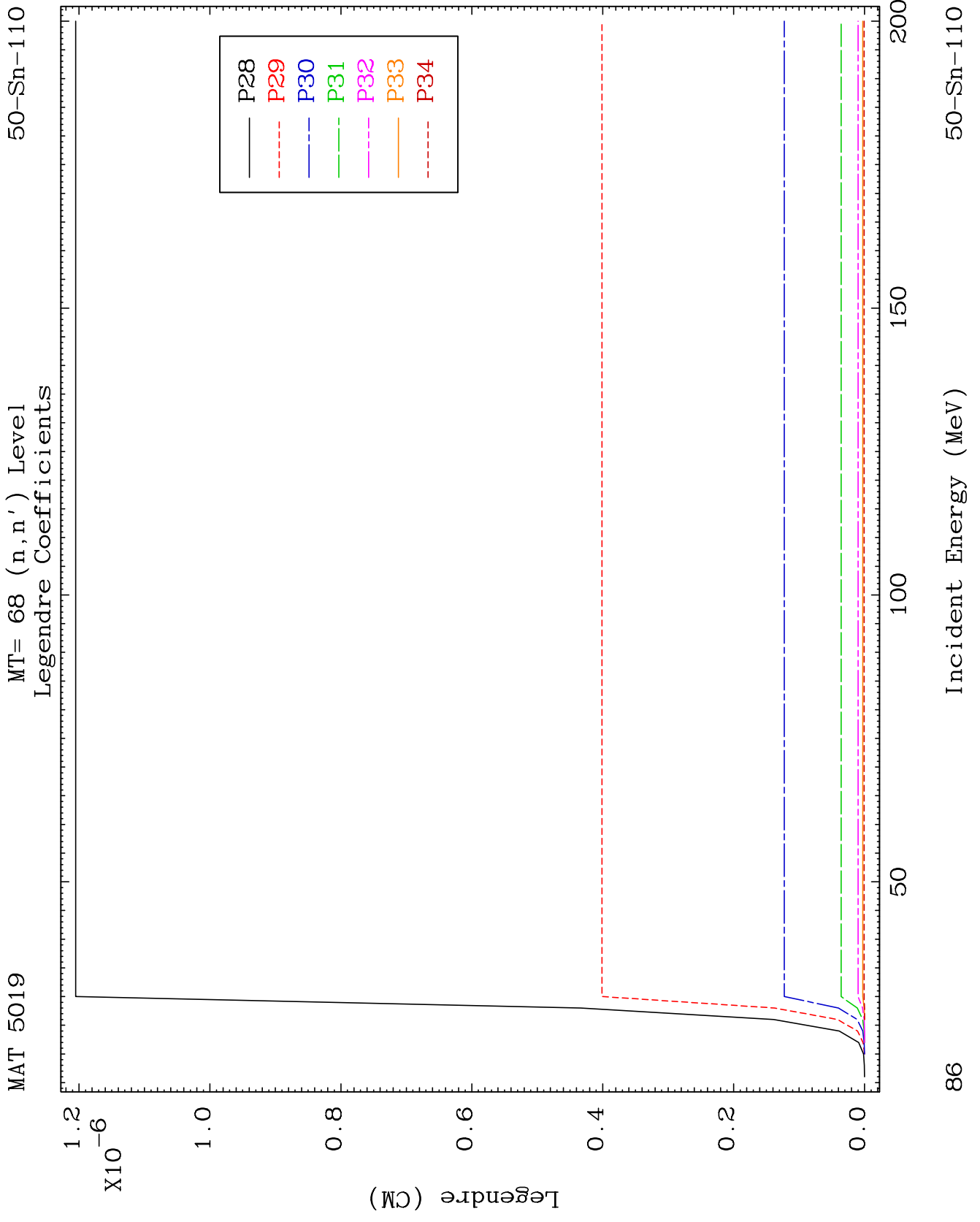
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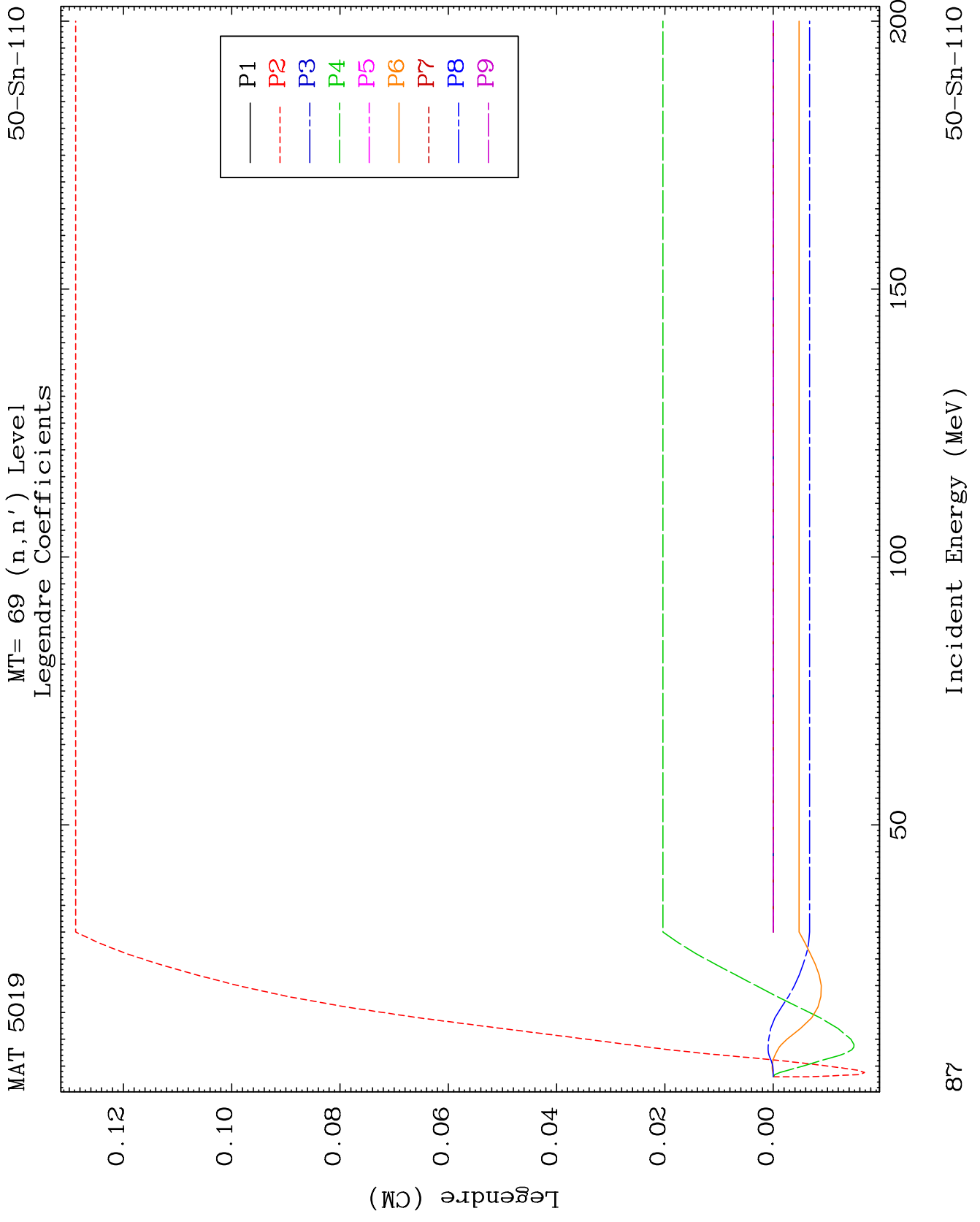
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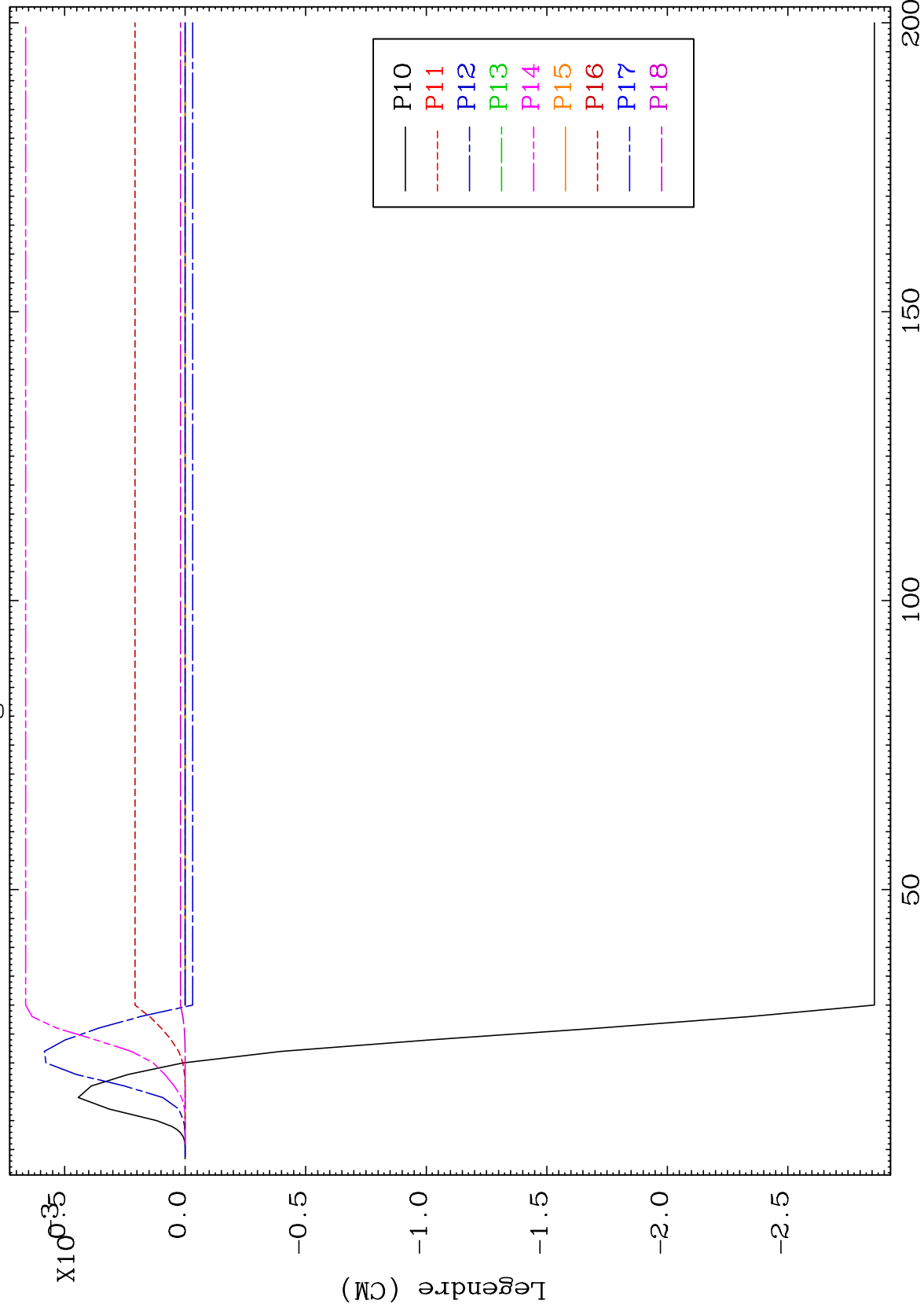




MAT 5019

MT= 69 (n,n') Level
Legendre Coefficients

50-Sn-110

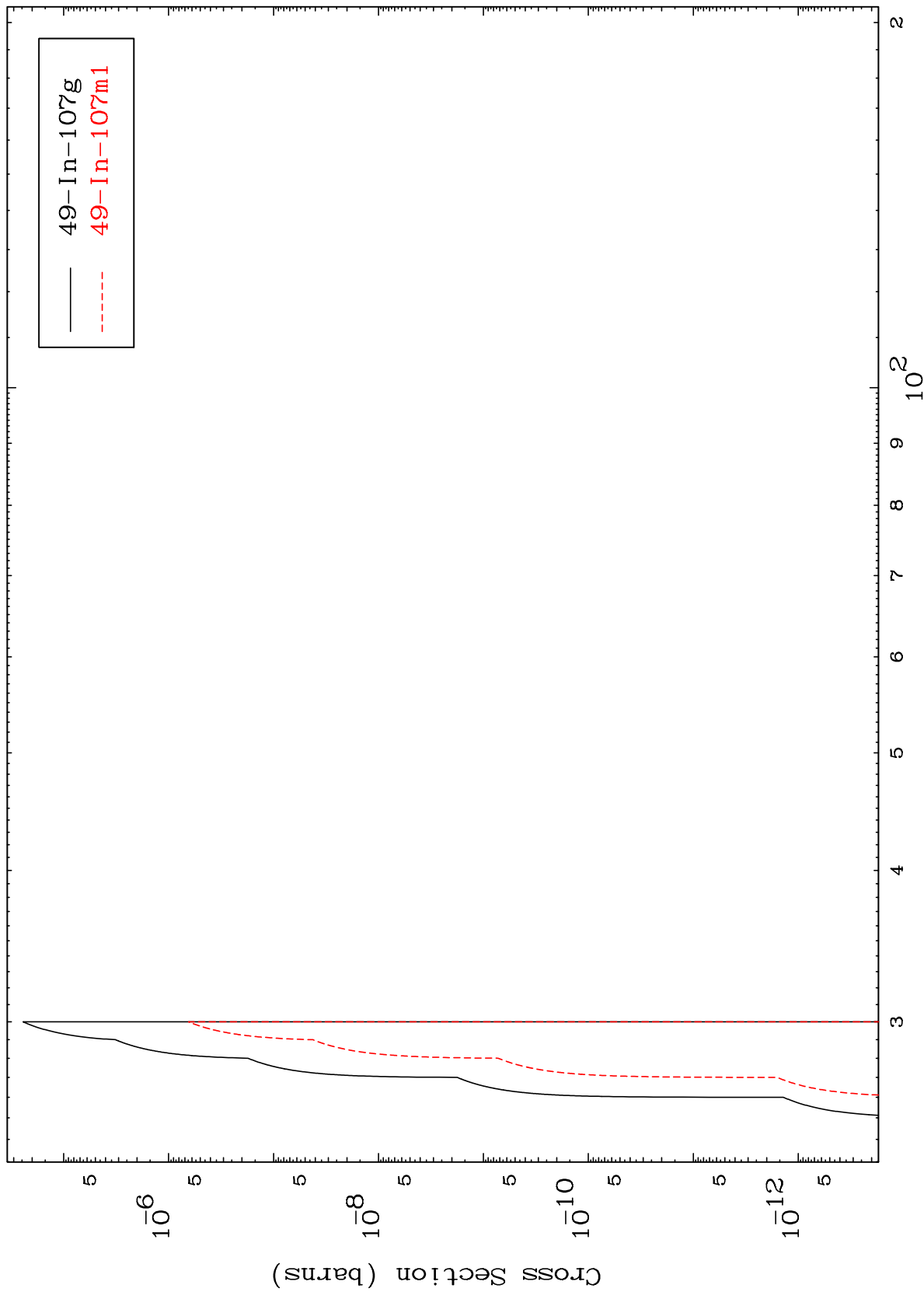


MAT 5019

50-Sn-110

(n,2n) d

Radionuclide Production Cross Section



50-Sn-110

Incident Energy (MeV)

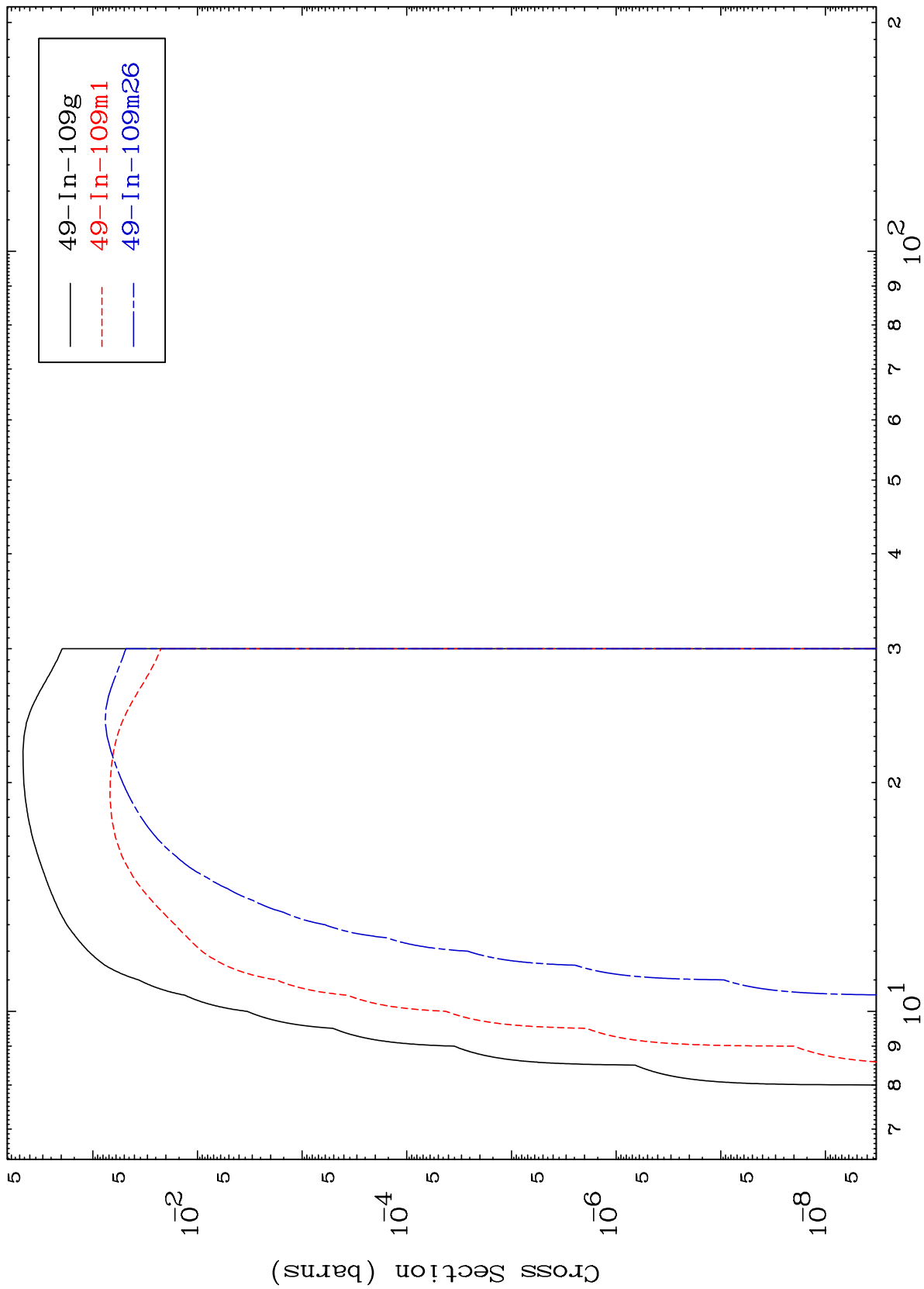
90

MAT 5019

(n,n') p

50-Sn-110

Radionuclide Production Cross Section



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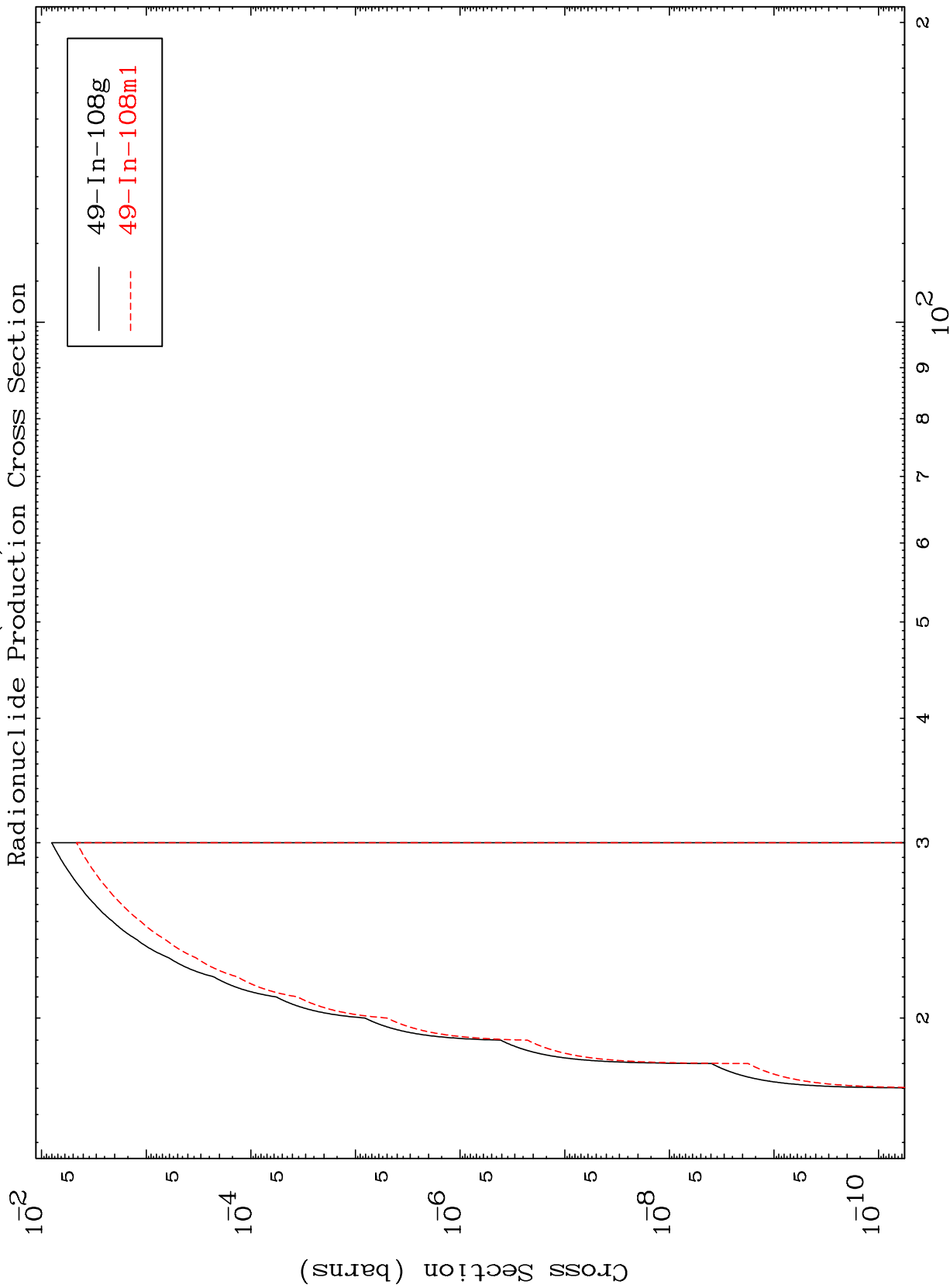
Incident Energy (MeV)

50-Sn-110

MAT 5019

(n,n') d

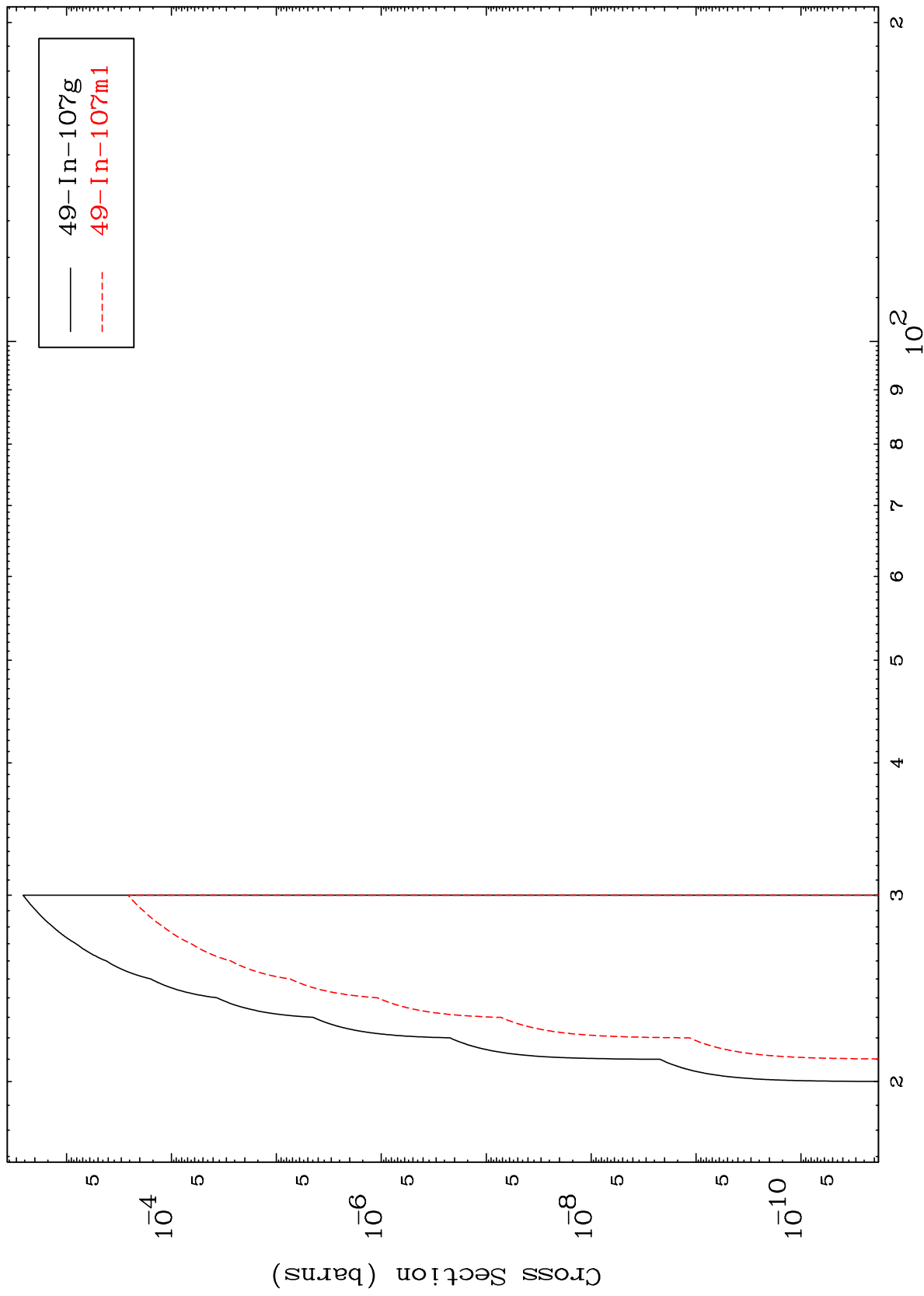
50-Sn-110



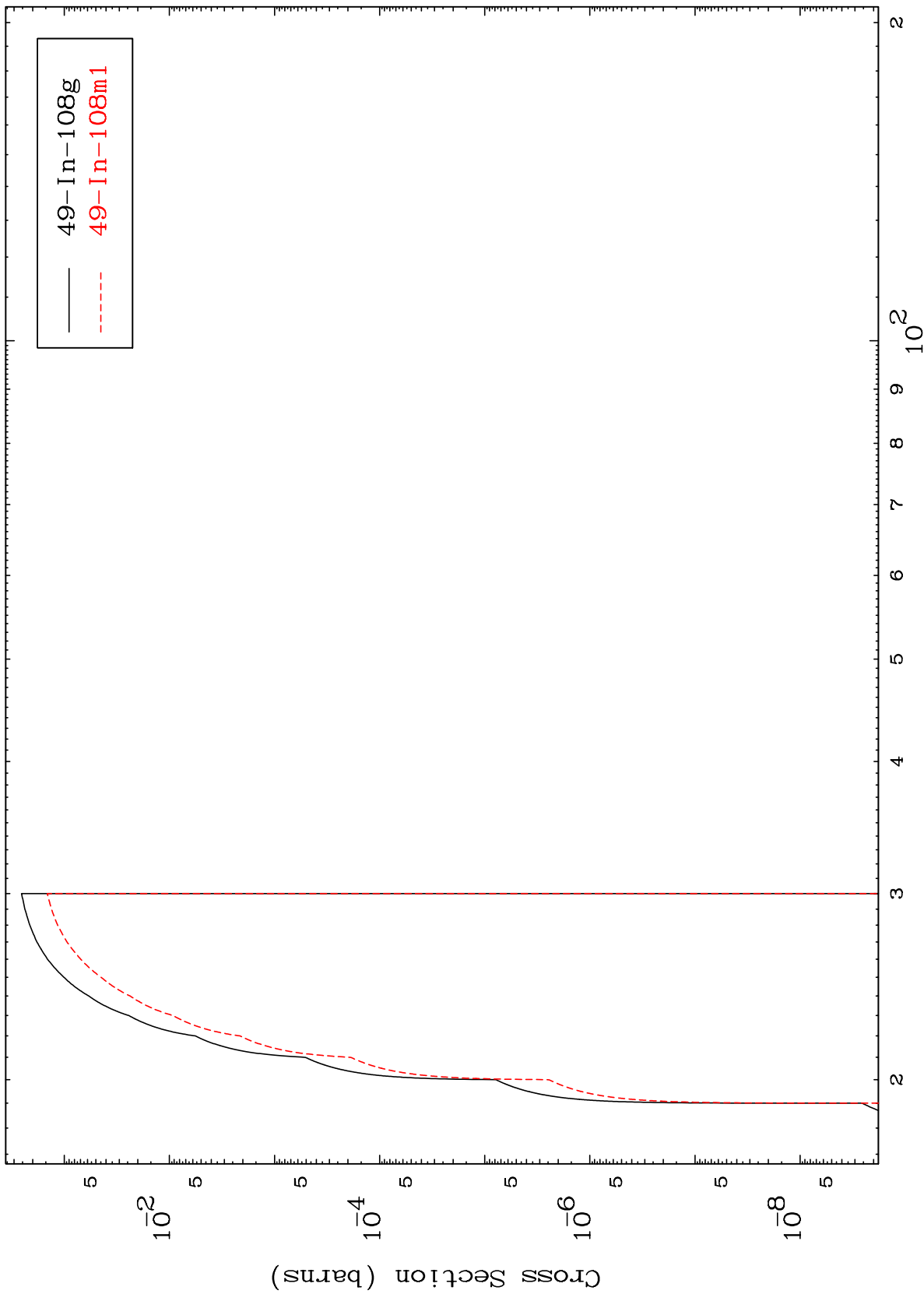
92

50-Sn-110

Radionuclide Production Cross Section



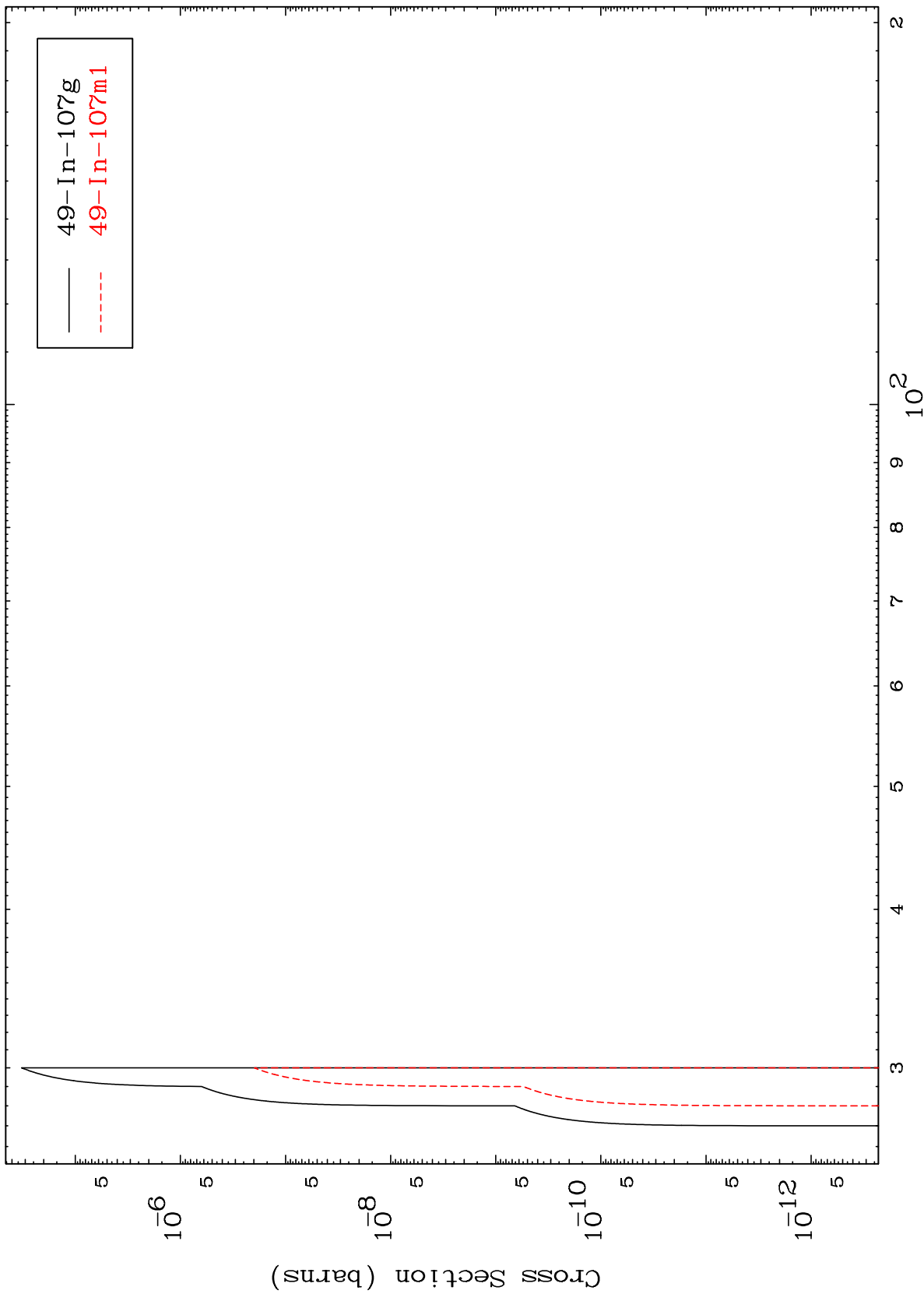
Radionuclide Production Cross Section



MAT 5019

50-Sn-110

(n,3n) p
Radionuclide Production Cross Section



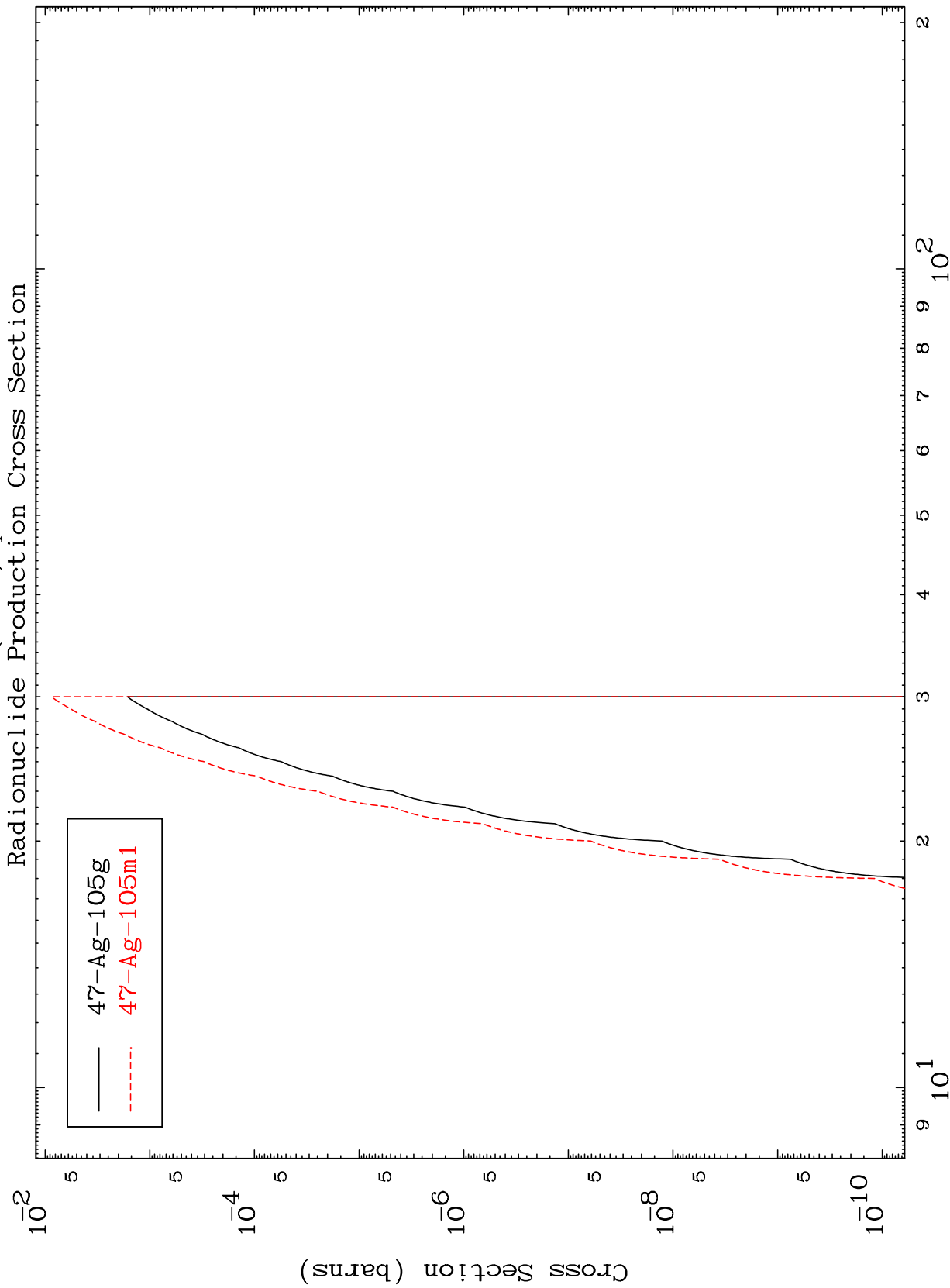
95

50-Sn-110

MAT 5019

(n,n') p α

50-Sn-110

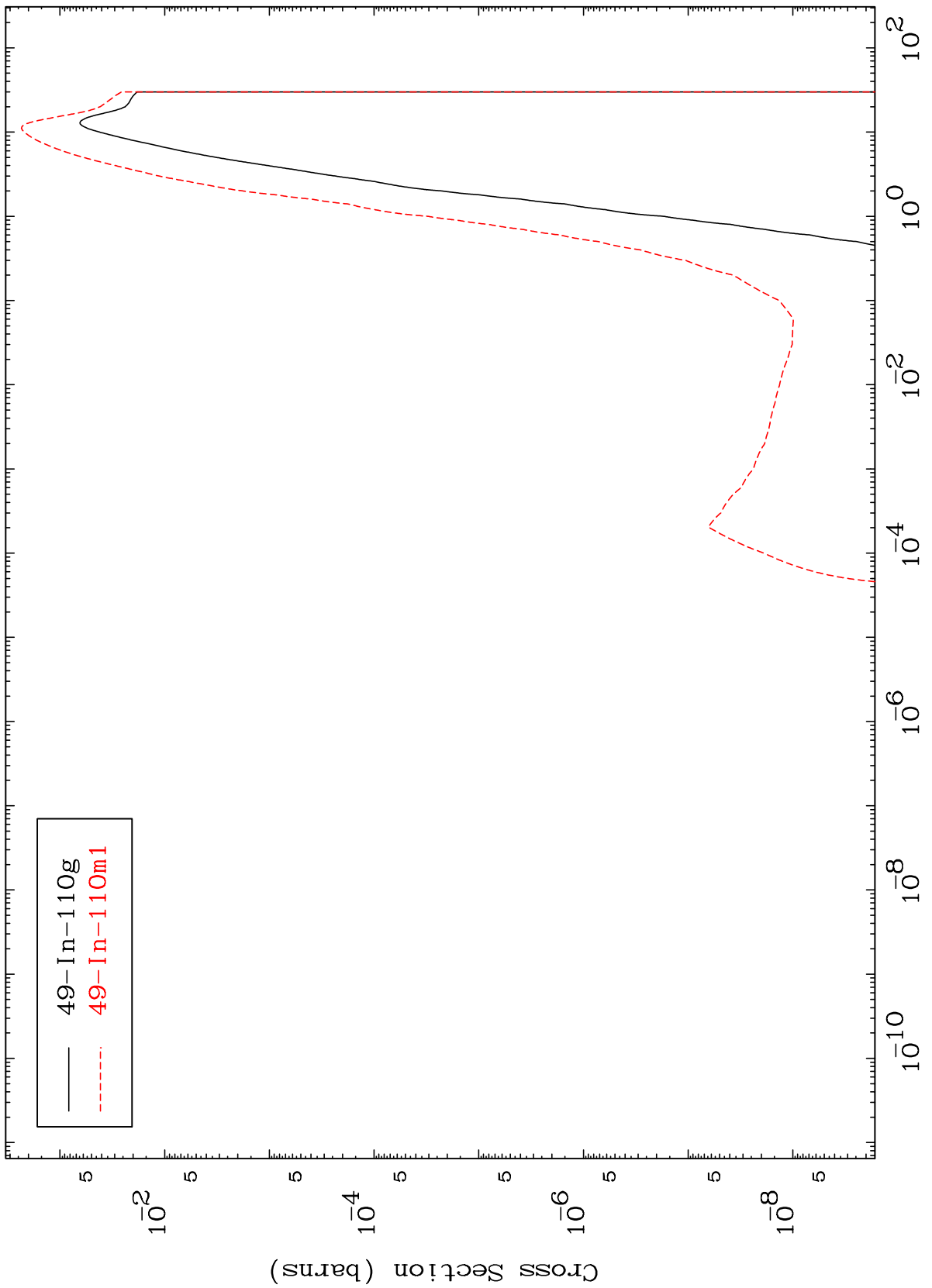


47-Ag-105g
47-Ag-105m1

MAT 5019

50-Sn-110

(n,p)
Radionuclide Production Cross Section

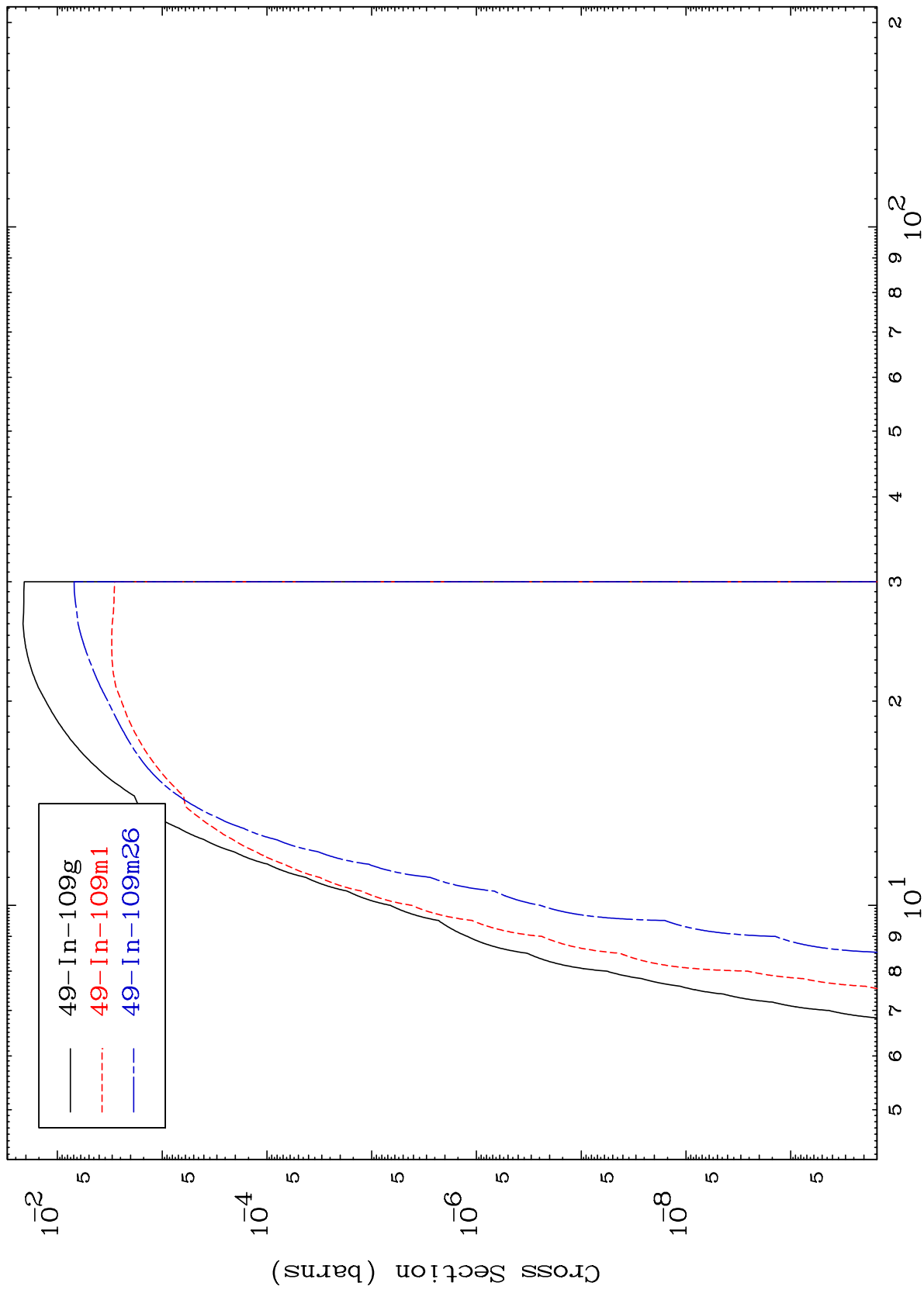


50-Sn-110

MAT 5019

50-Sn-110

(n,d)
Radionuclide Production Cross Section



98

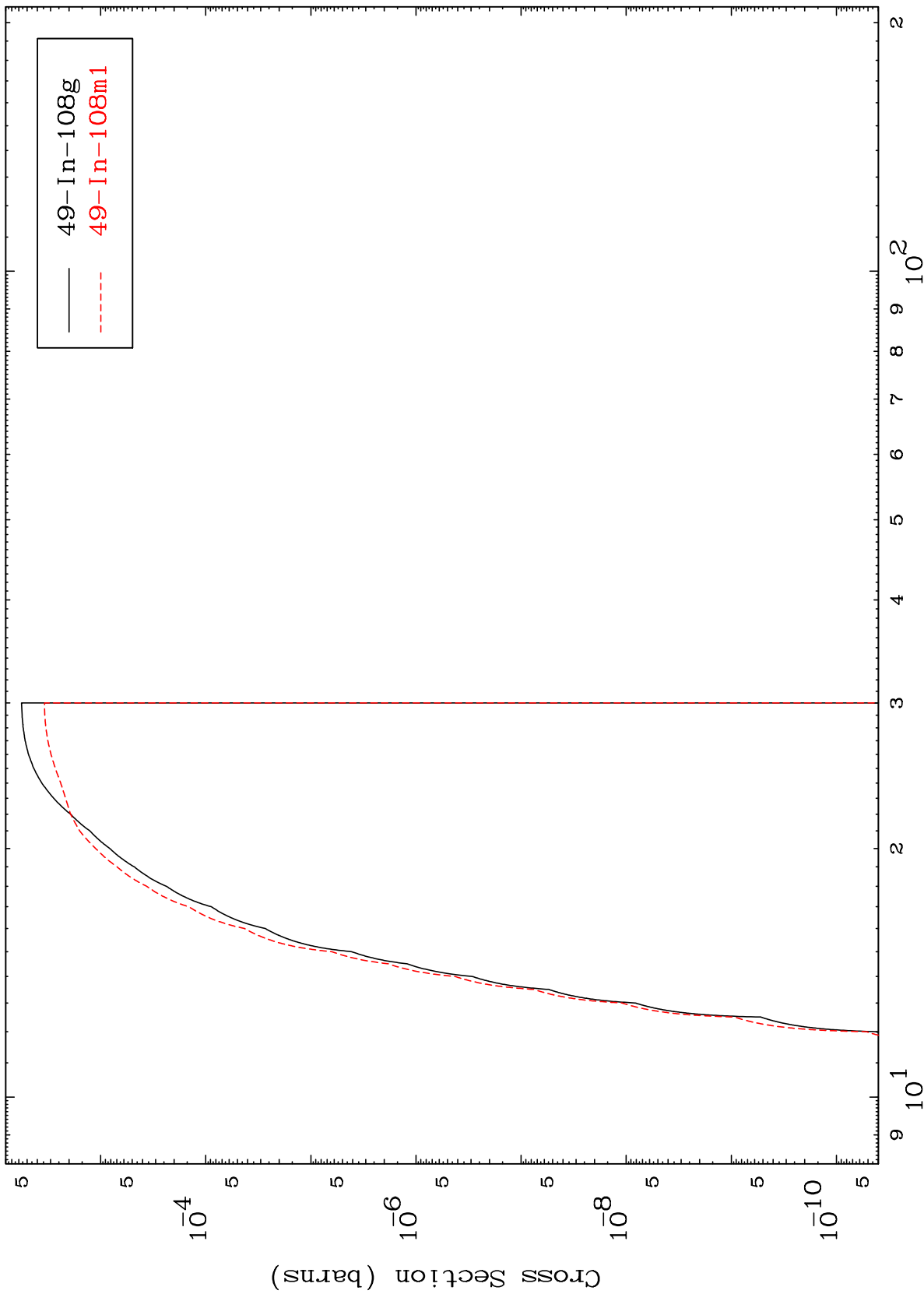
Incident Energy (MeV)

50-Sn-110

MAT 5019

50-Sn-110

(n,t)
Radionuclide Production Cross Section



99

Incident Energy (MeV)

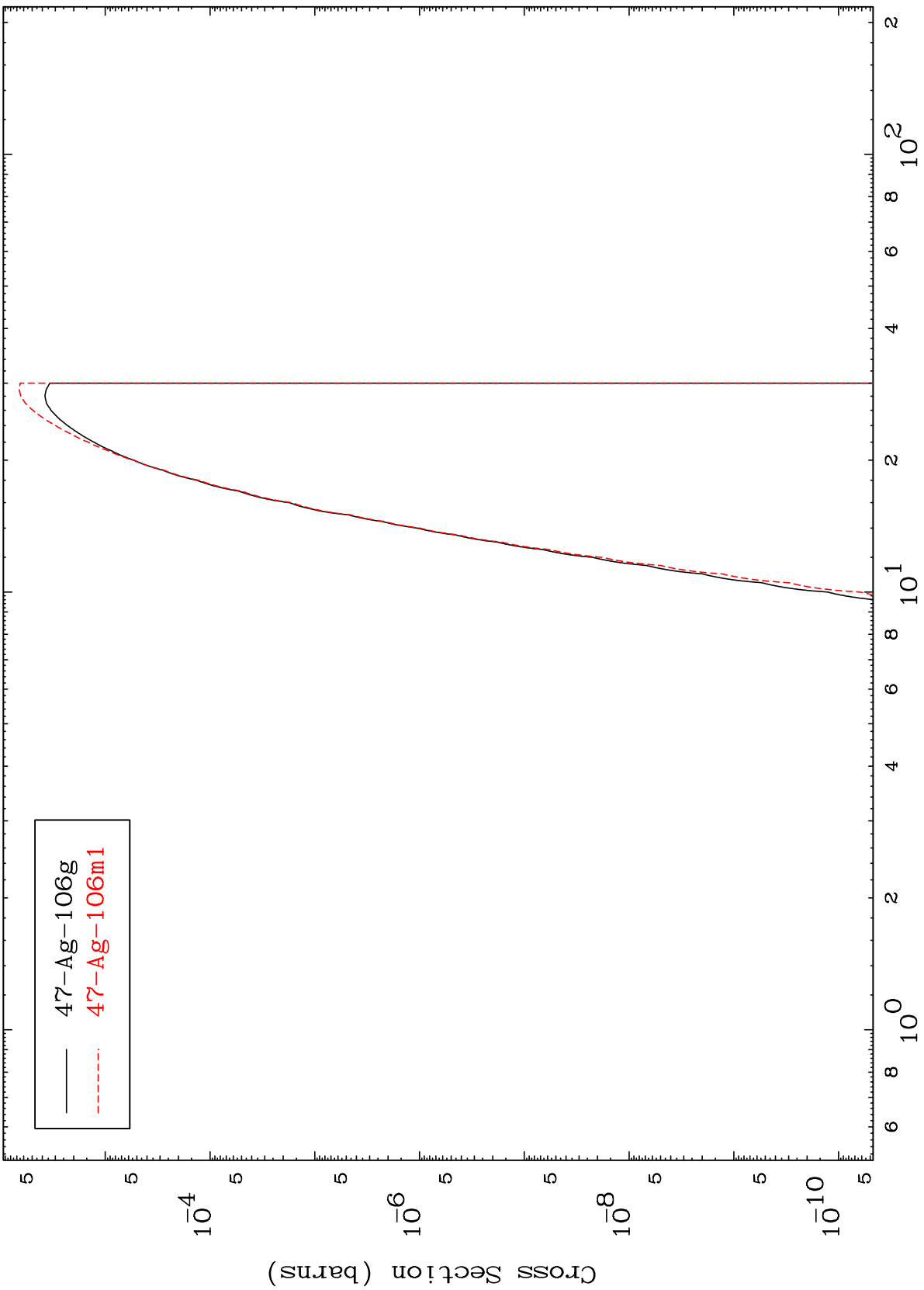
50-Sn-110

MAT 5019

(n,p) α

50-Sn-110

Radionuclide Production Cross Section



100

Incident Energy (MeV)

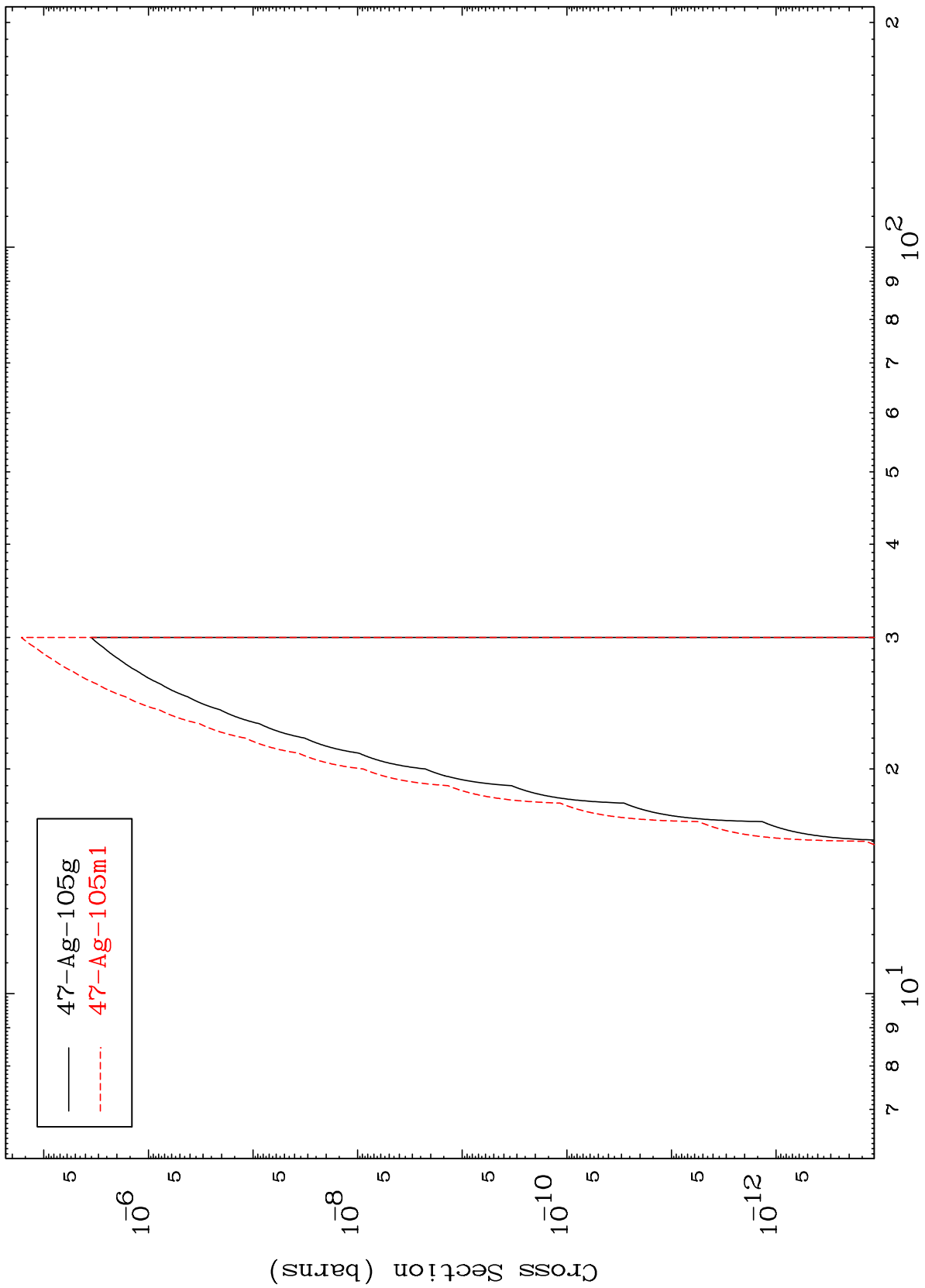
50-Sn-110

MAT 5019

50-Sn-110

(n,d) α

Radionuclide Production Cross Section



101

Incident Energy (MeV)

50-Sn-110