

Program EVALPLOT  
(Version 2021-1)

by

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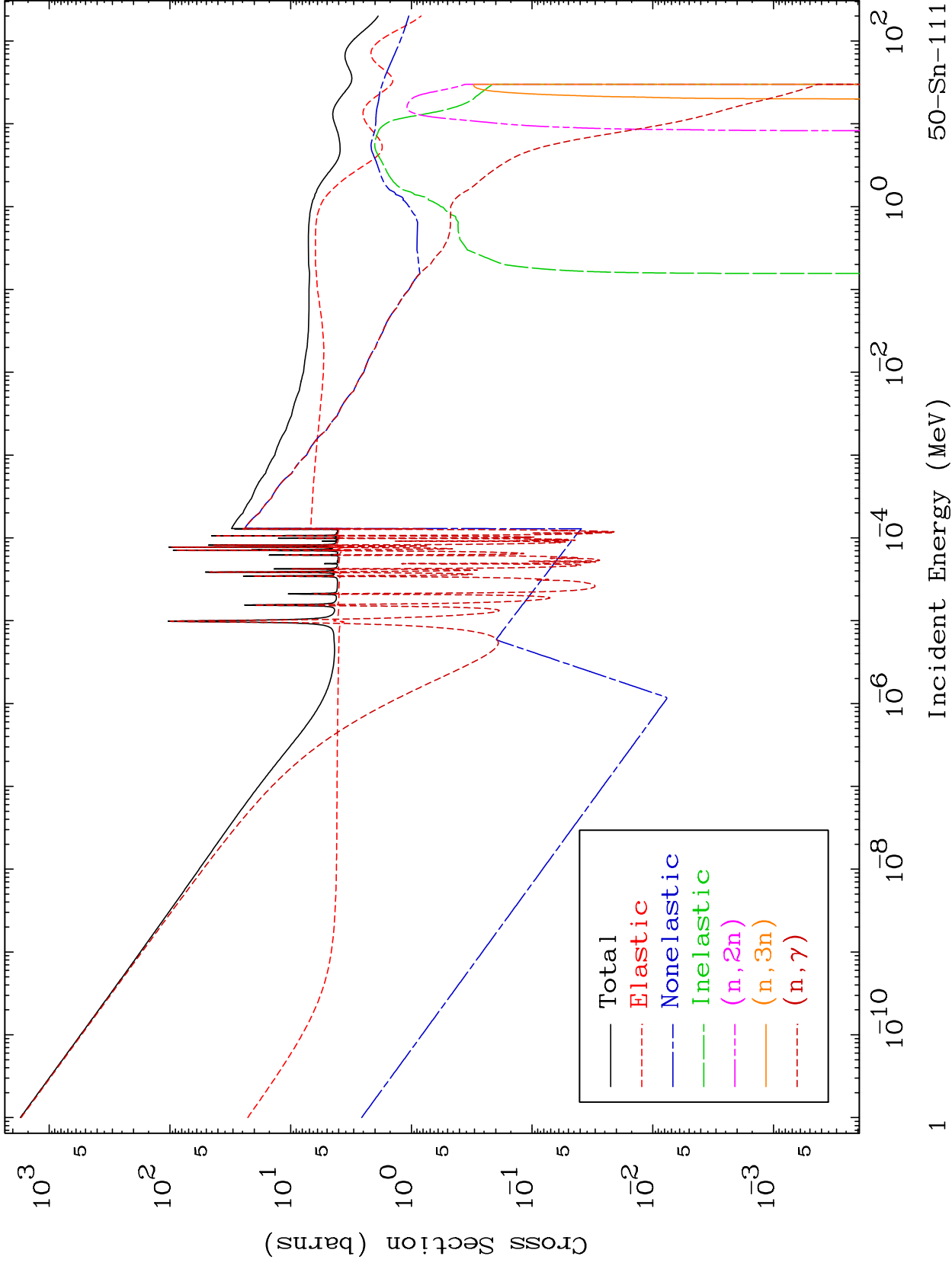
Web: [redcullen1.net/HOMEPAGE.NEW](http://redcullen1.net/HOMEPAGE.NEW)

Press Mouse Button to Start

MAT 5022

Neutron Major  
293 Kelvin Cross Sections

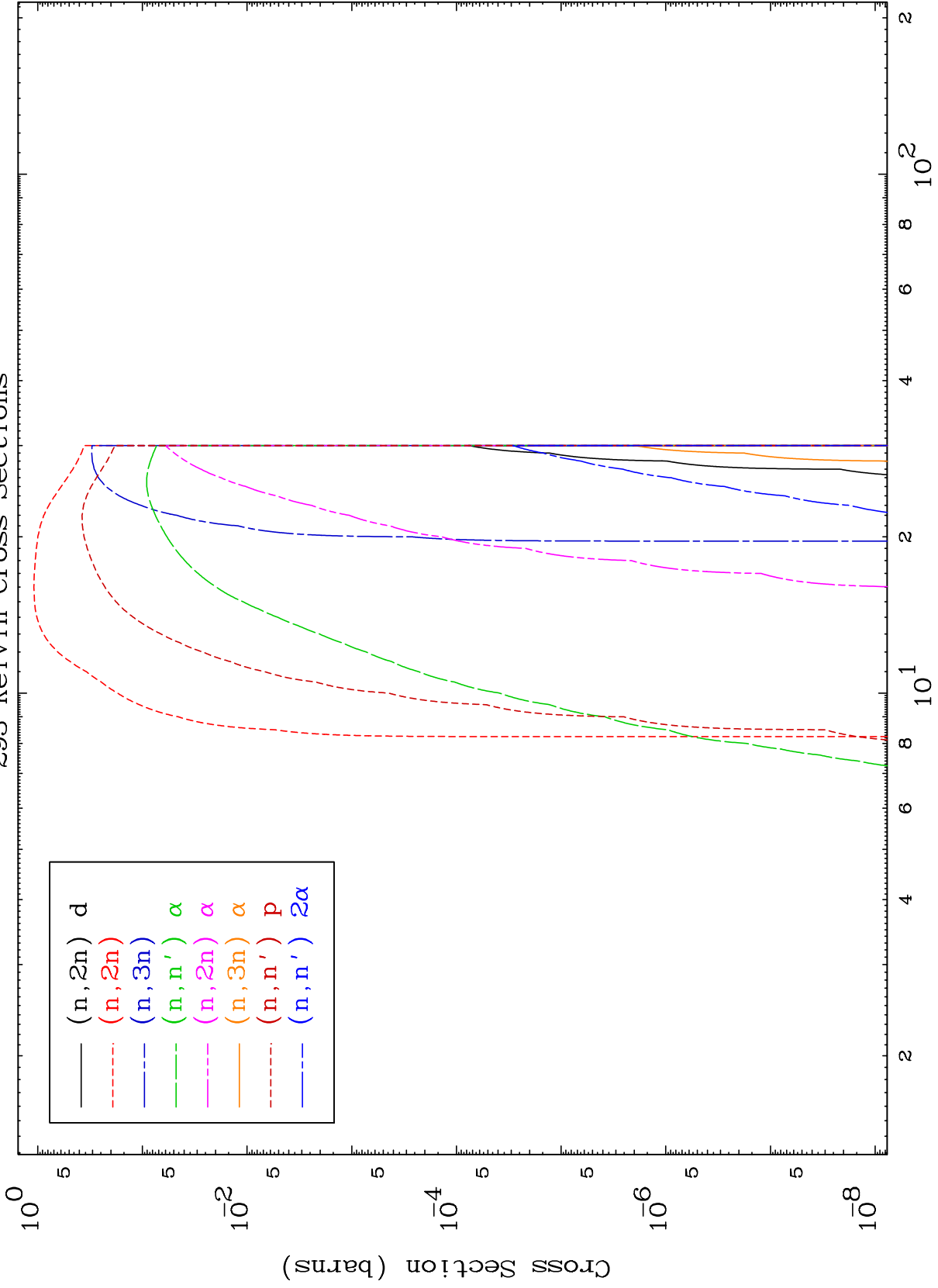
50-Sn-111



MAT 5022

Neutron Absorption  
293 Kelvin Cross Sections

50-Sn-111



2

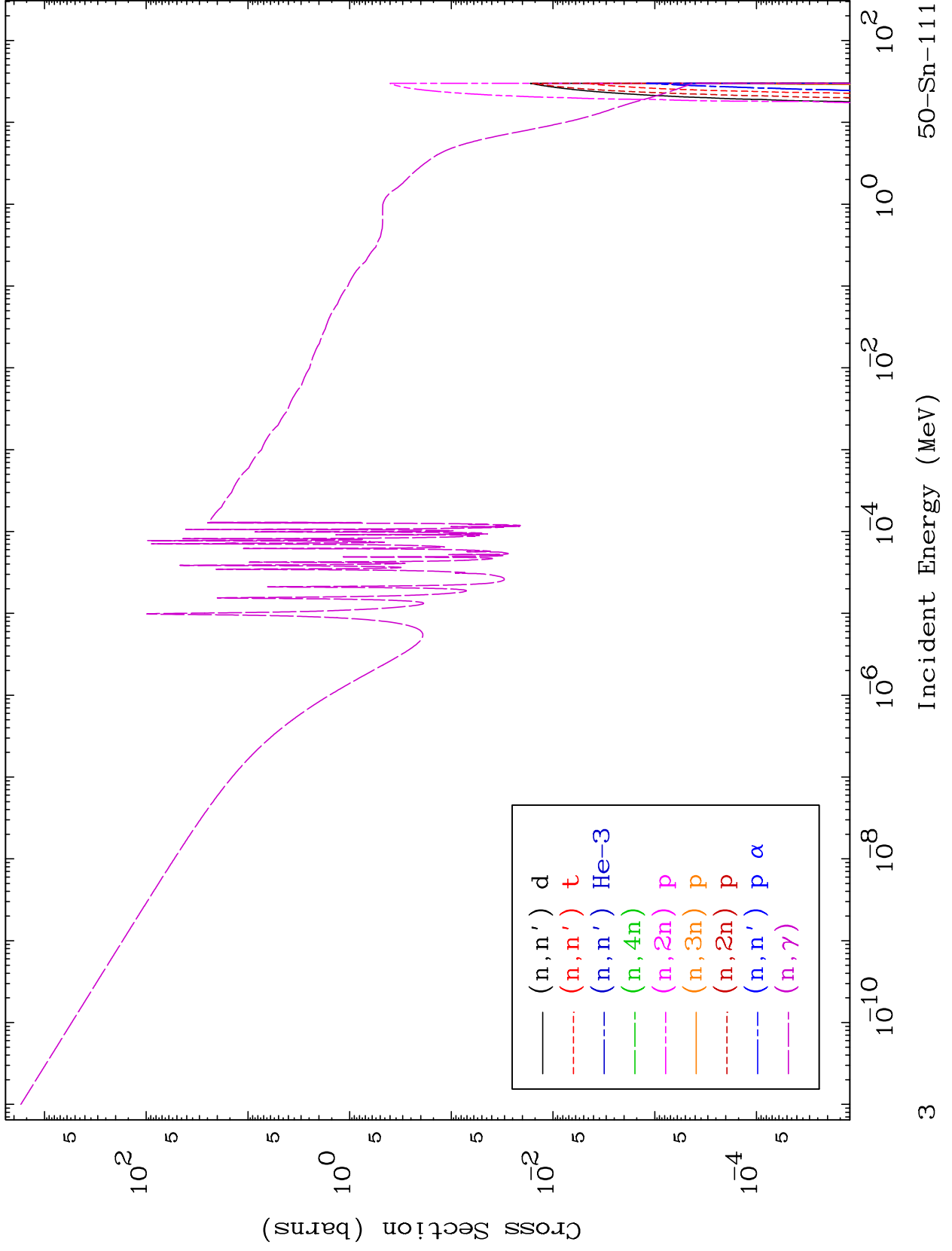
Incident Energy (MeV)

50-Sn-111

MAT 5022

Neutron Absorption  
293 Kelvin Cross Sections

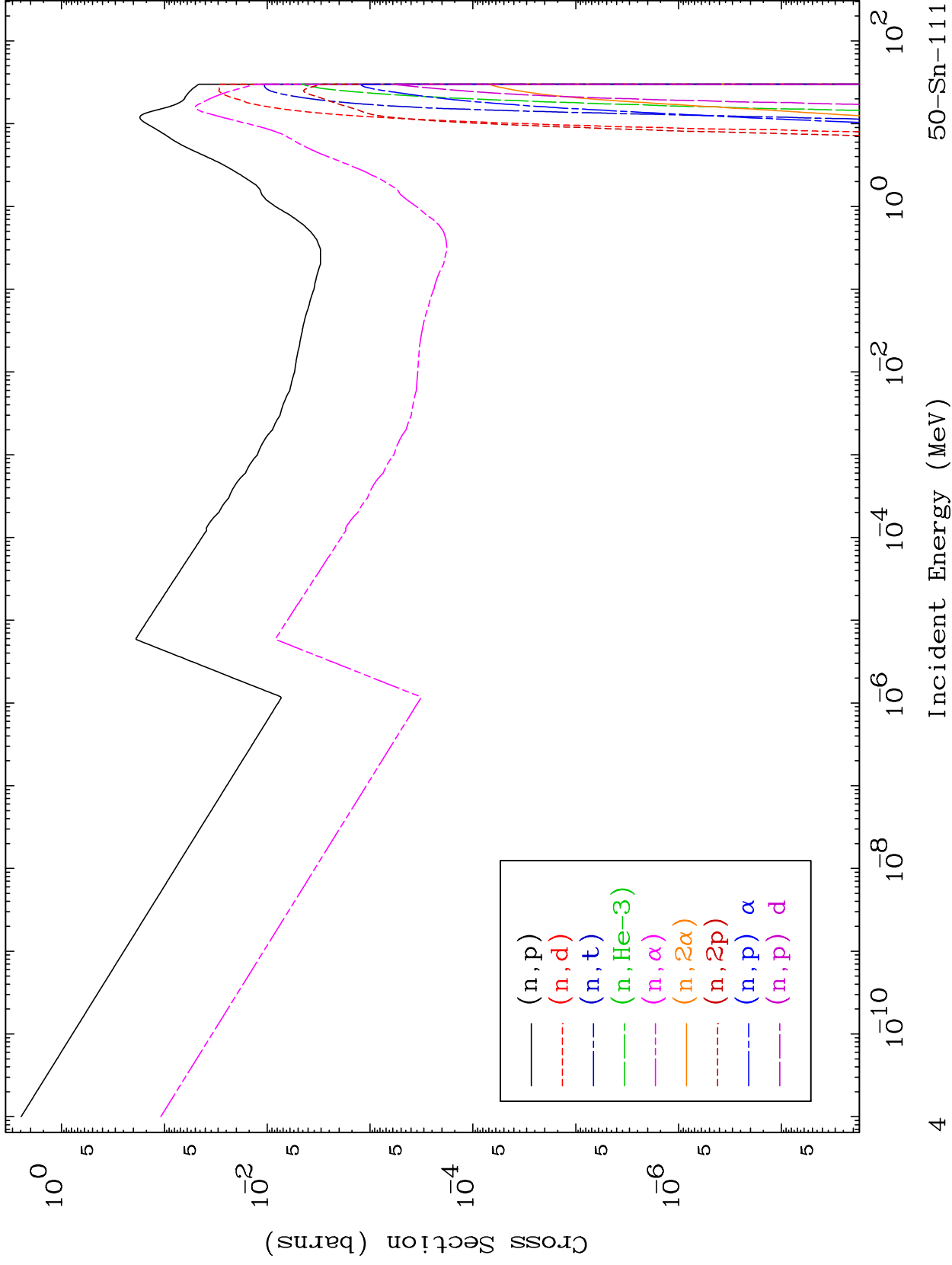
50-Sn-111

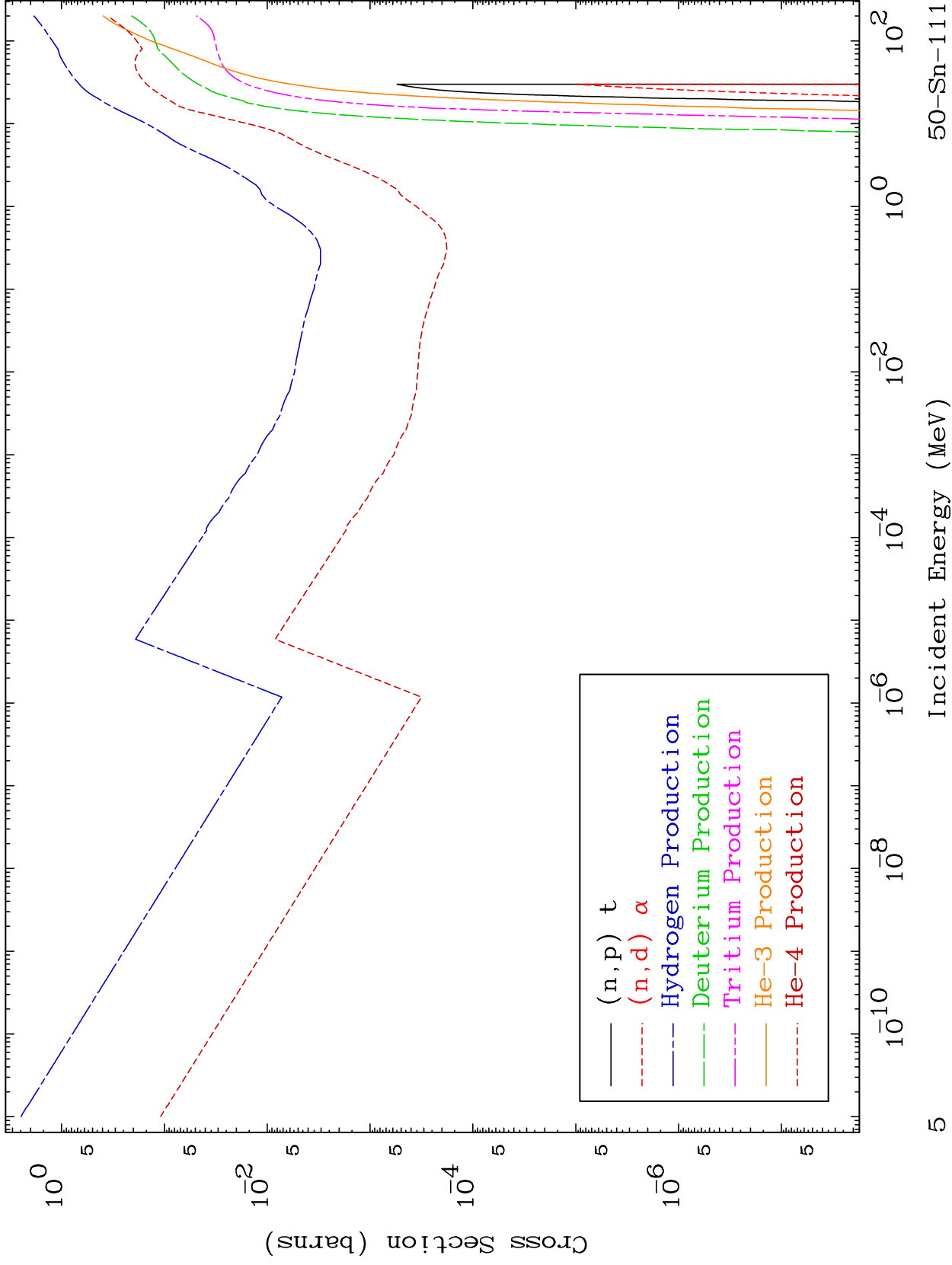


MAT 5022

Neutron Absorption  
293 Kelvin Cross Sections

50-Sn-111

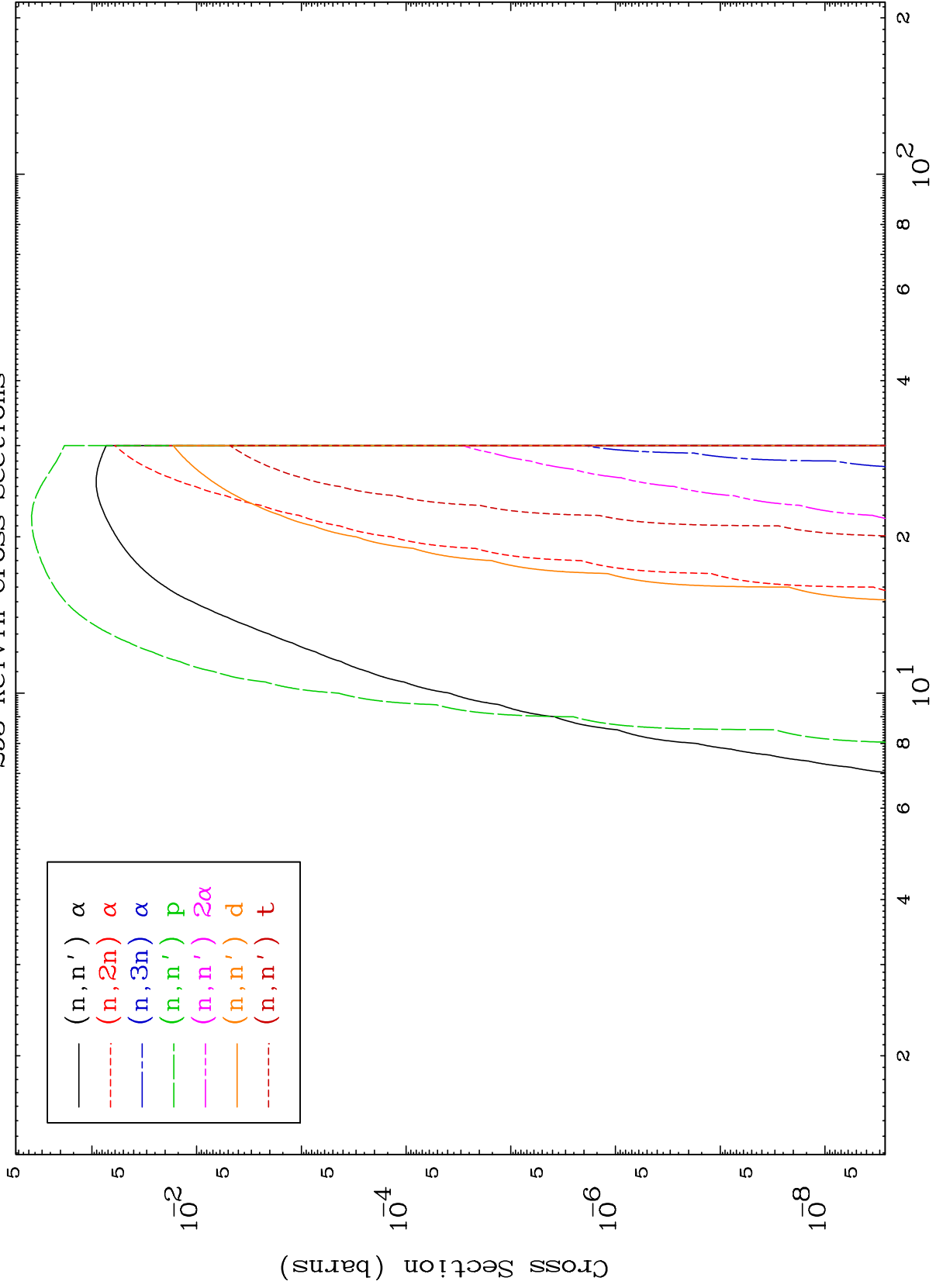




MAT 5022

Charged Particle  
293 Kelvin Cross Sections

50-Sn-111



6

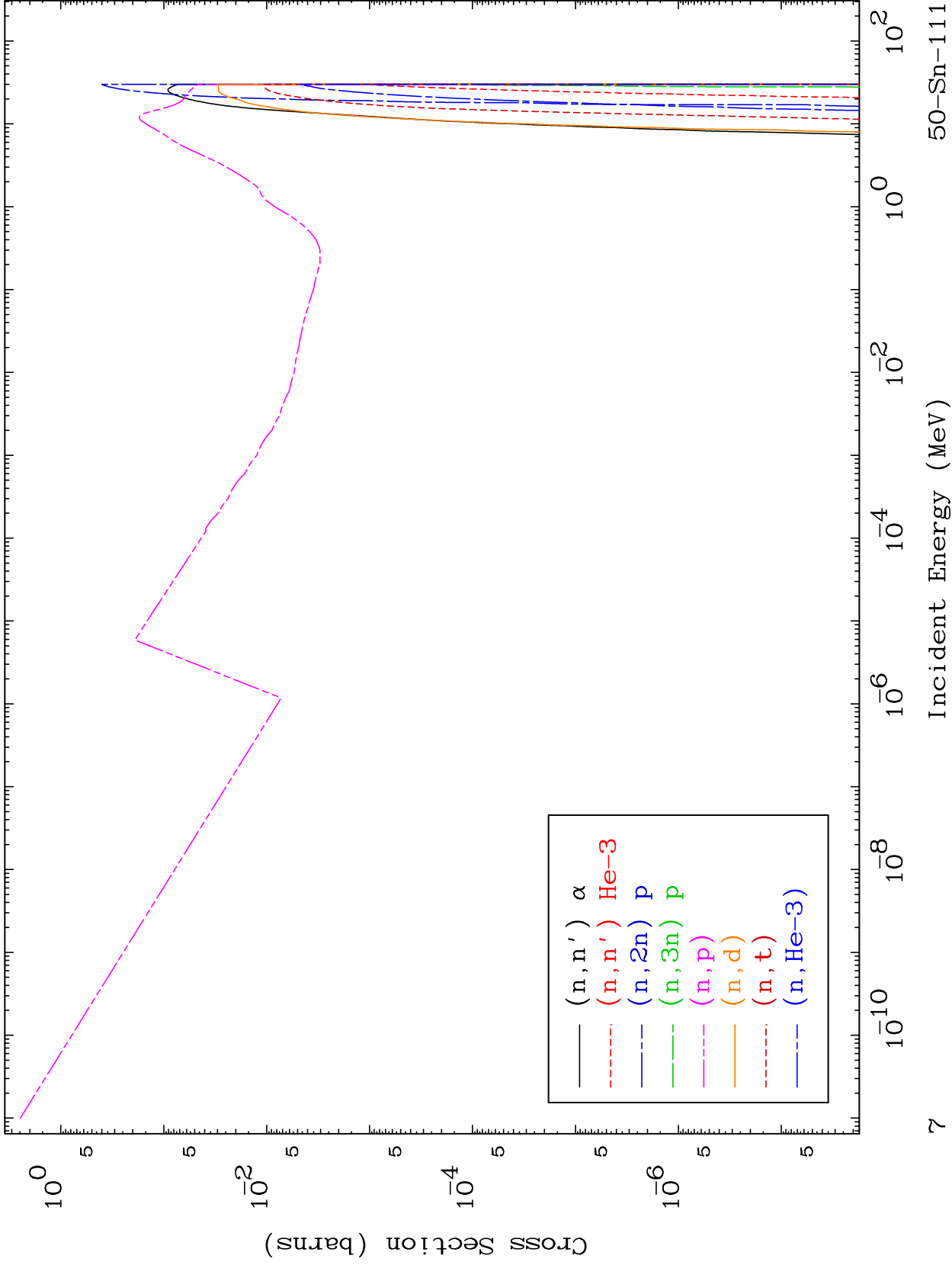
Incident Energy (MeV)

50-Sn-111

MAT 5022

Charged Particle  
293 Kelvin Cross Sections

50-Sn-111

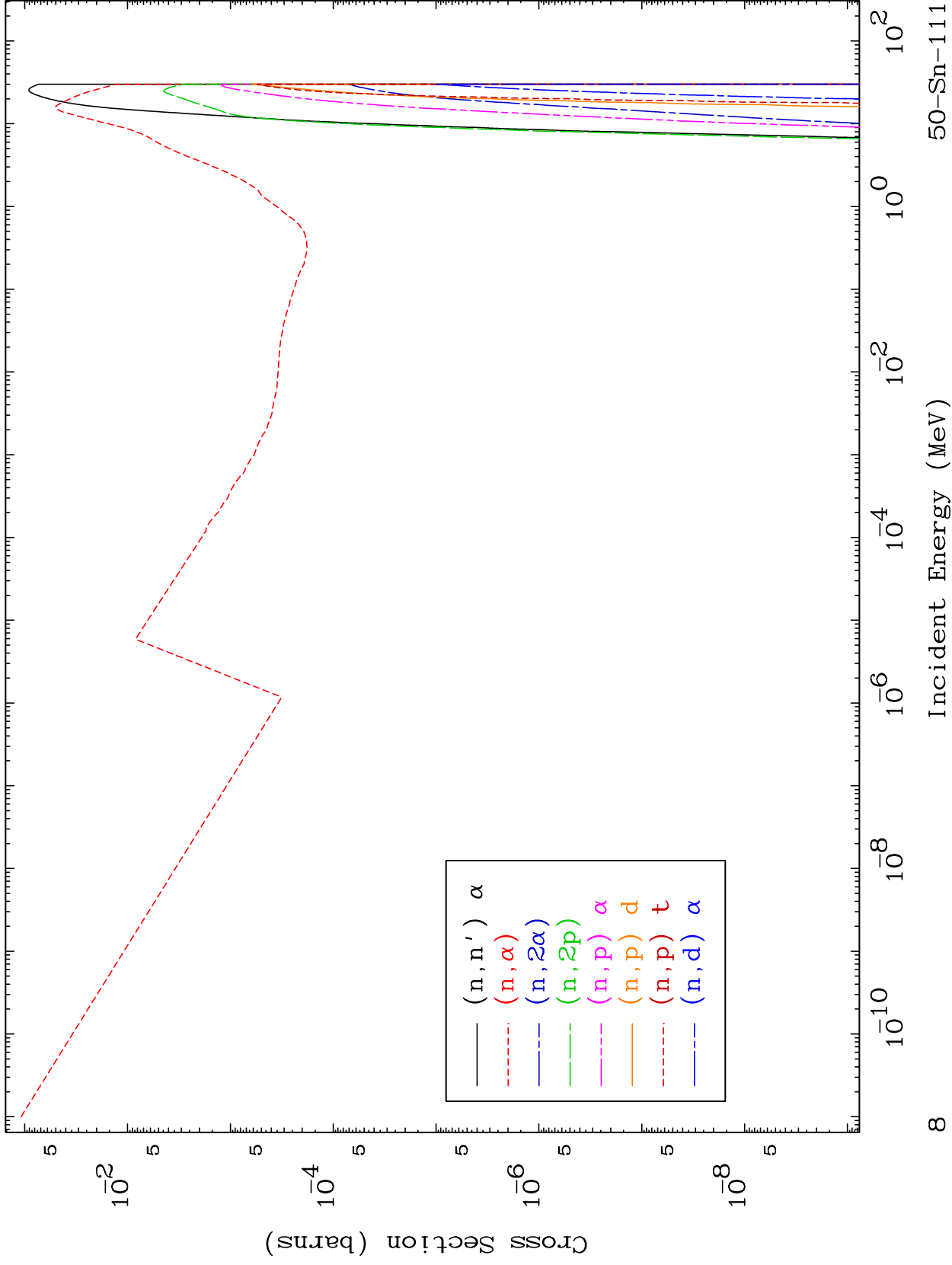




MAT 5022

Charged Particle  
293 Kelvin Cross Sections

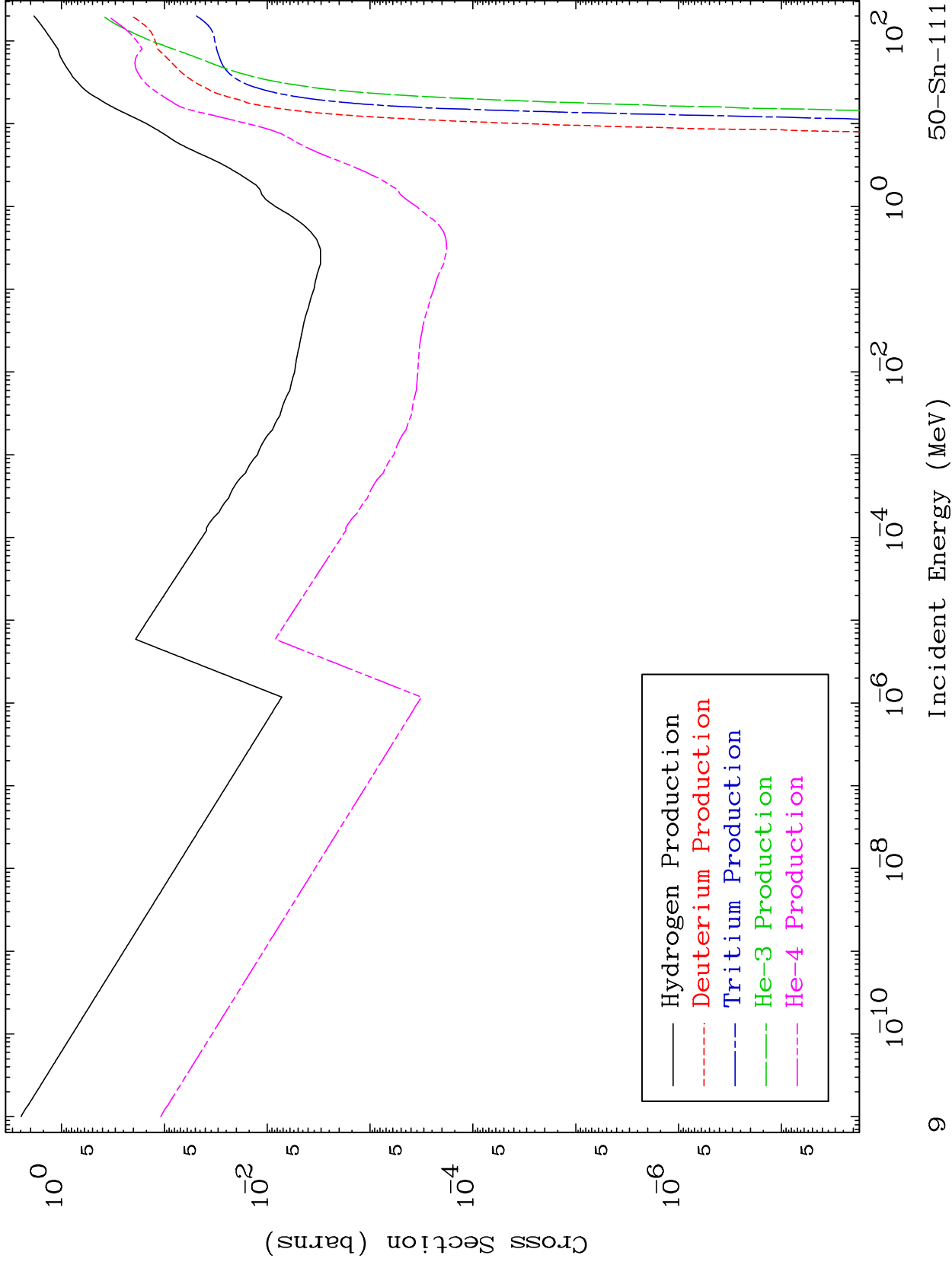
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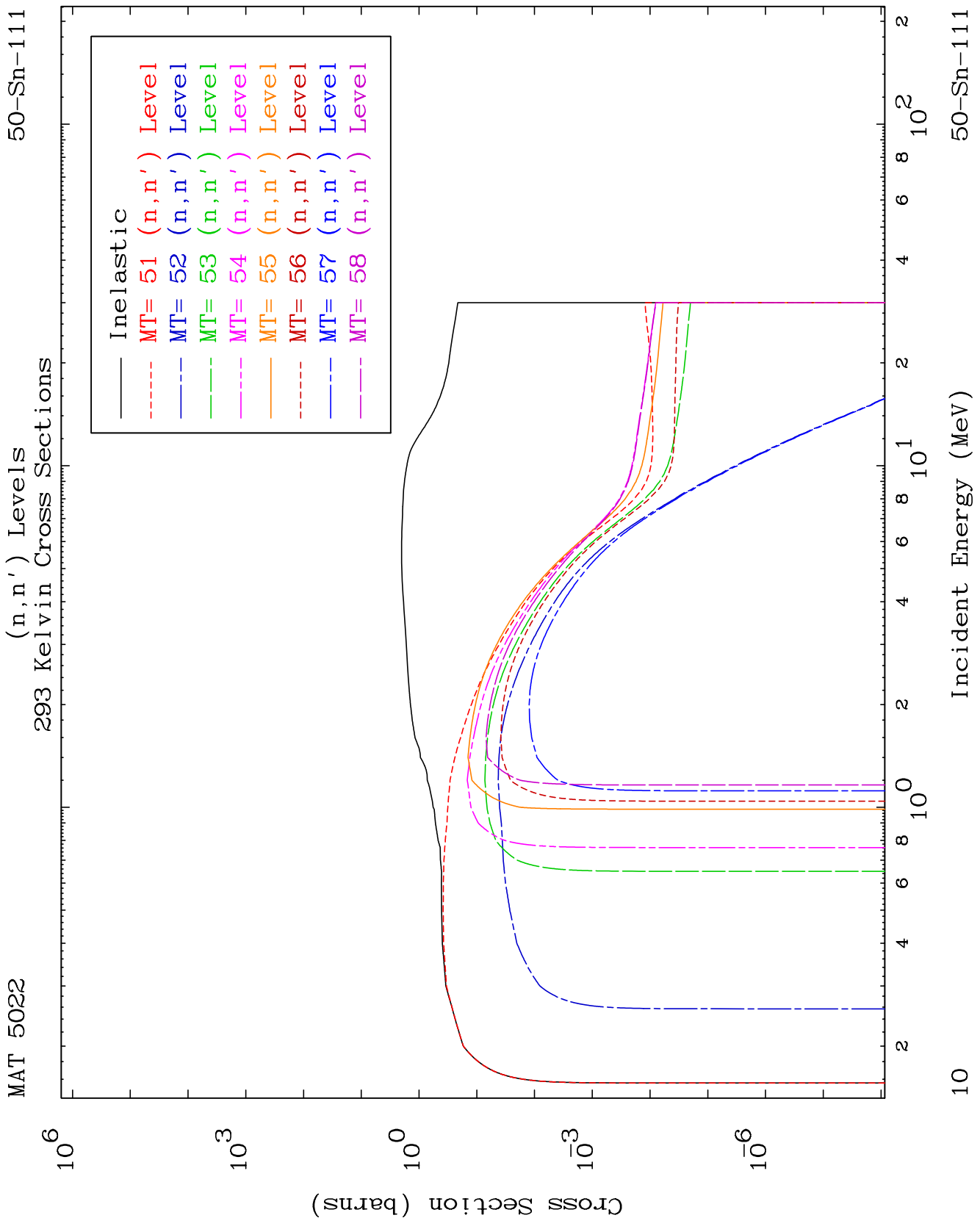


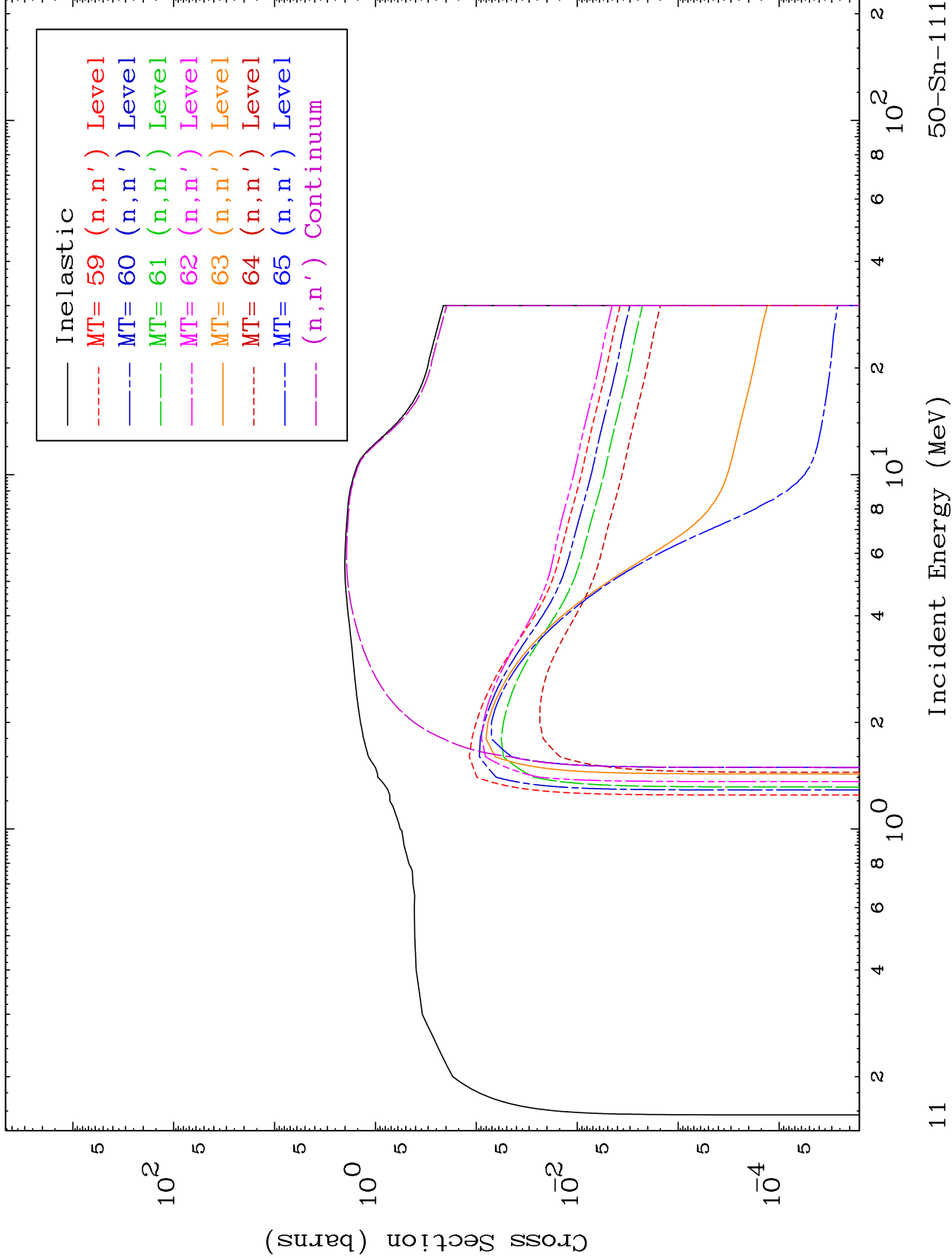
MAT 5022

Particle Production  
293 Kelvin Cross Sections

50-Sn-111



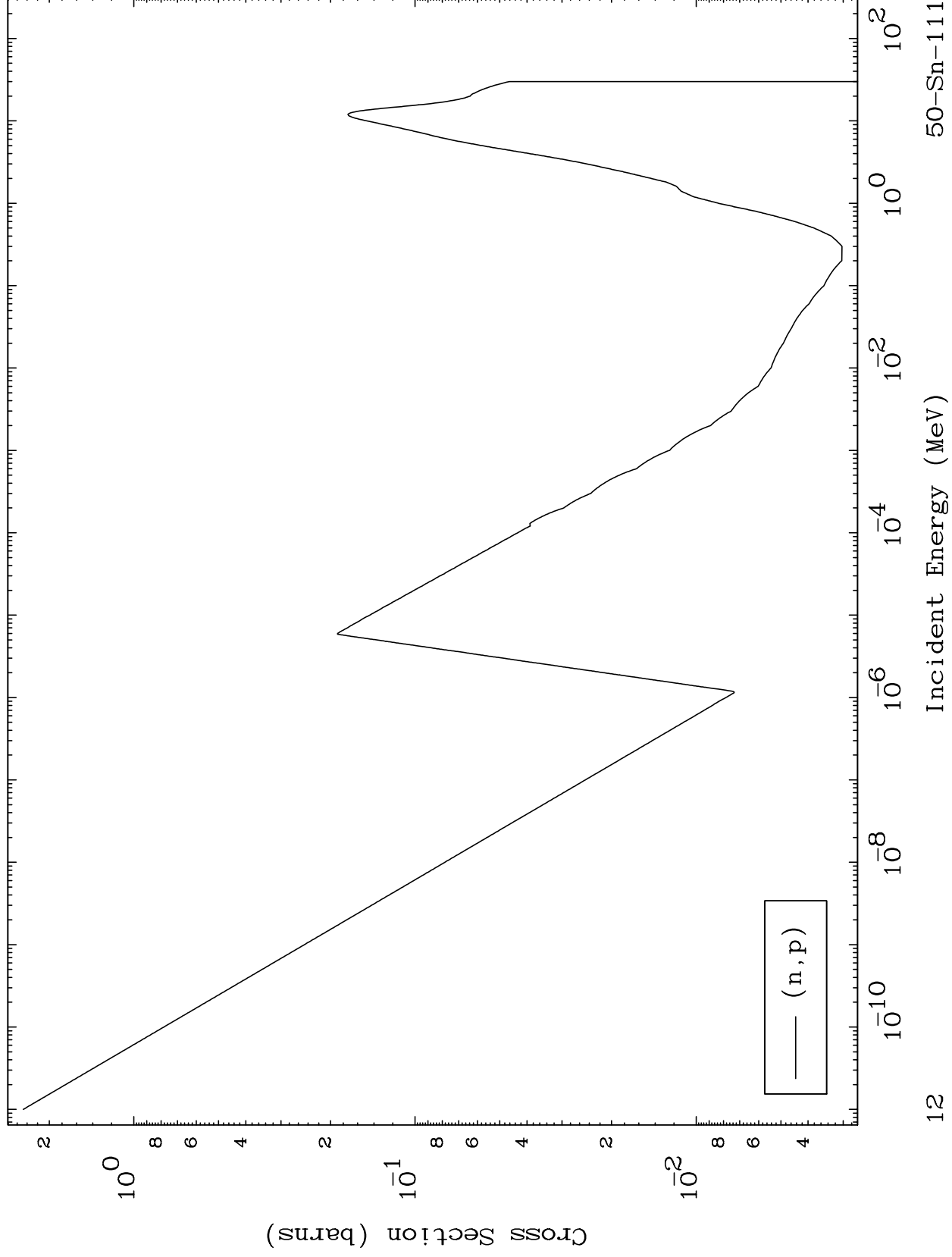




MAT 5022

(n,p) Levels  
293 Kelvin Cross Sections

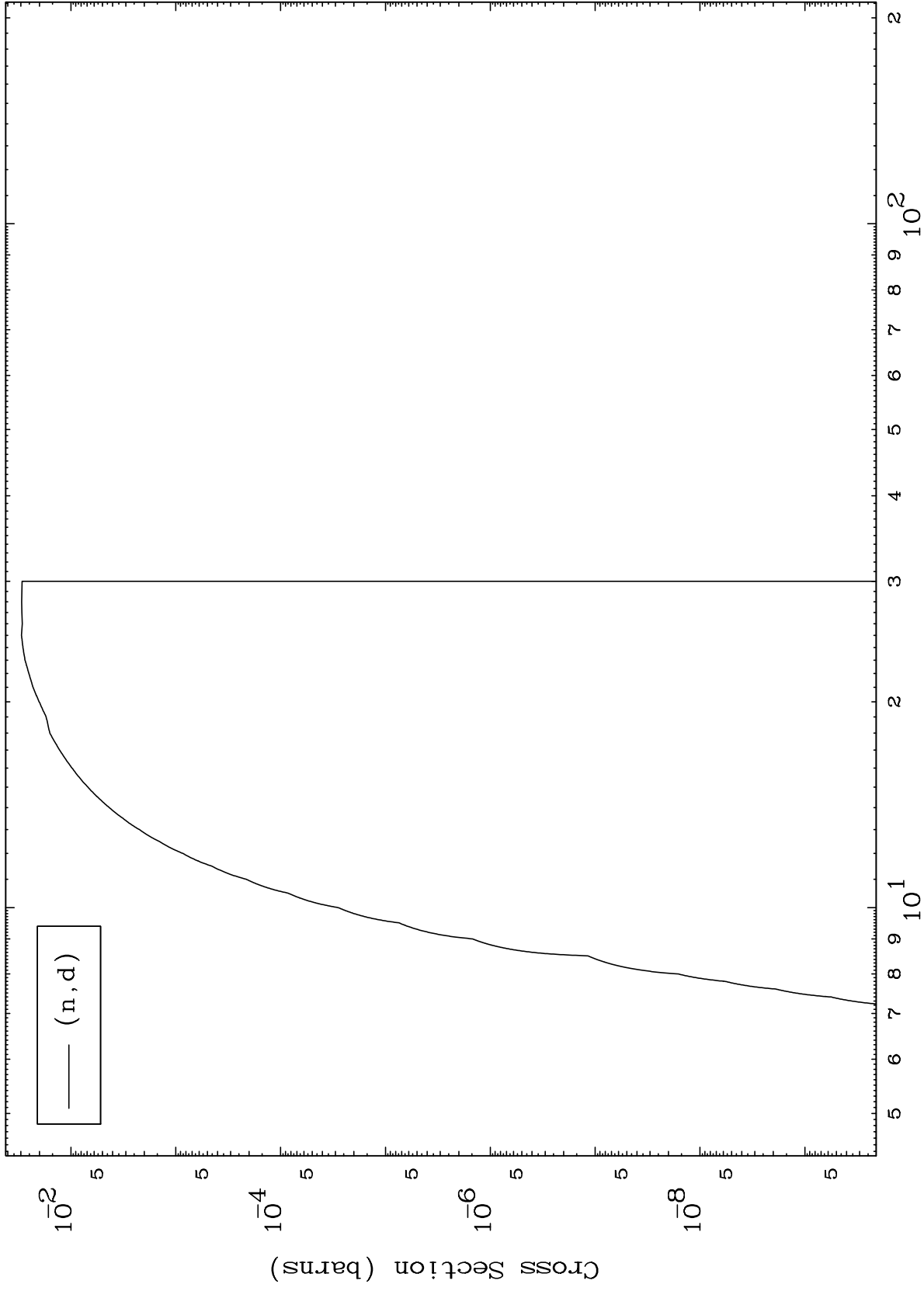
50-Sn-111



MAT 5022

(n,d) Levels  
293 Kelvin Cross Sections

50-Sn-111



13

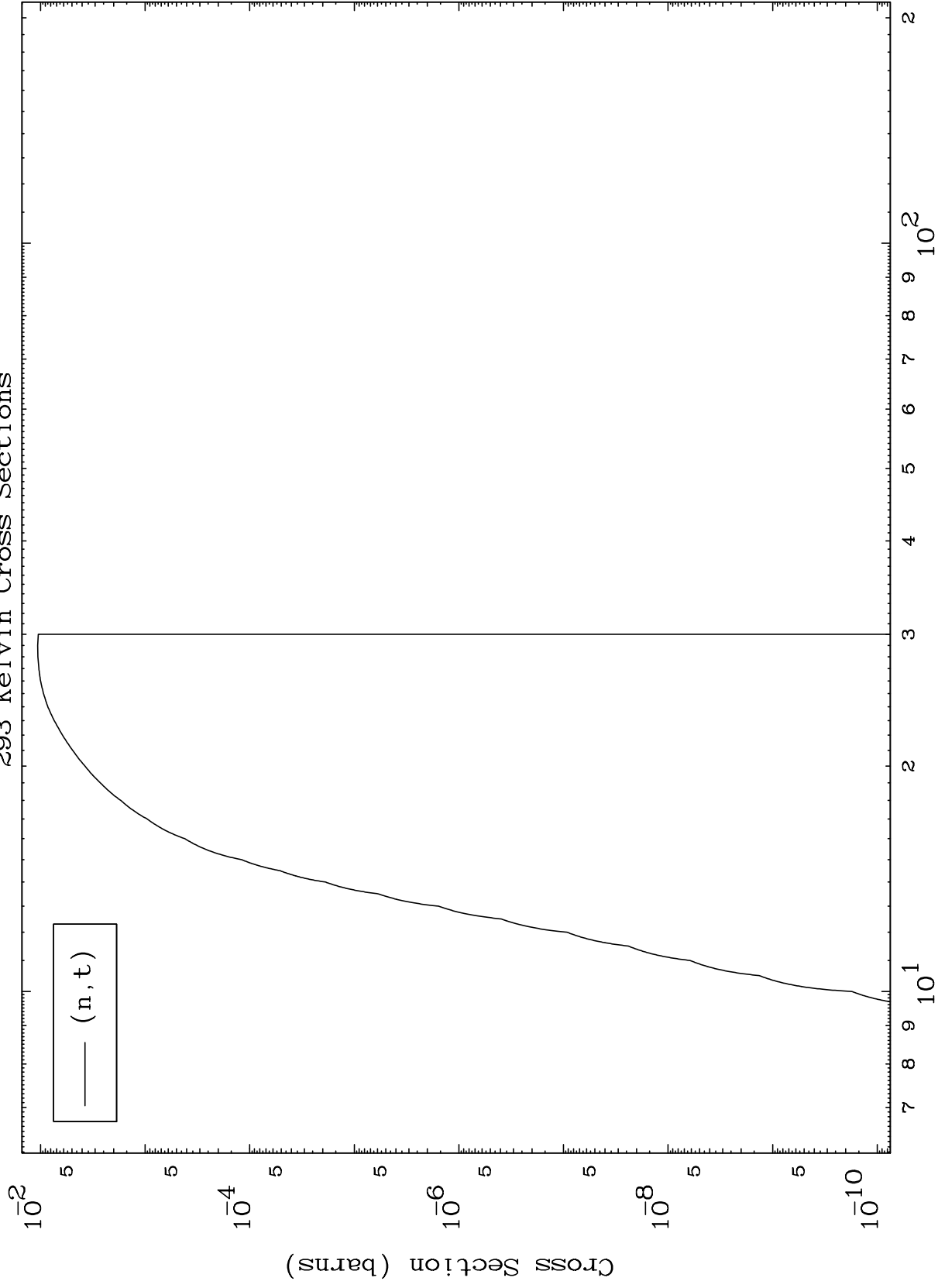
Incident Energy (MeV)

50-Sn-111

MAT 5022

(n,t) Levels  
293 Kelvin Cross Sections

50-Sn-111



14

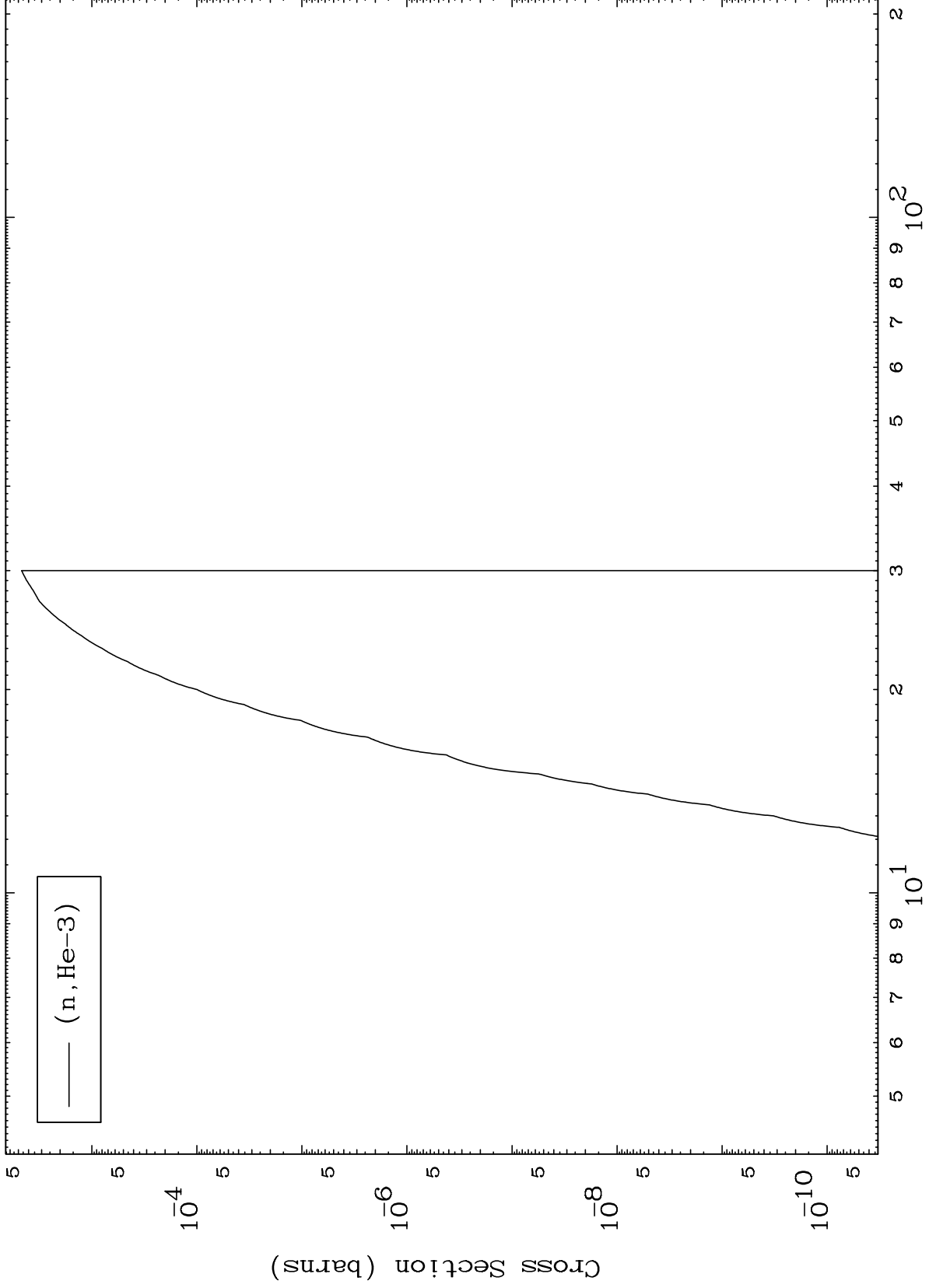
Incident Energy (MeV)

50-Sn-111

MAT 5022

(n,He3) Levels  
293 Kelvin Cross Sections

50-Sn-111



15

Incident Energy (MeV)

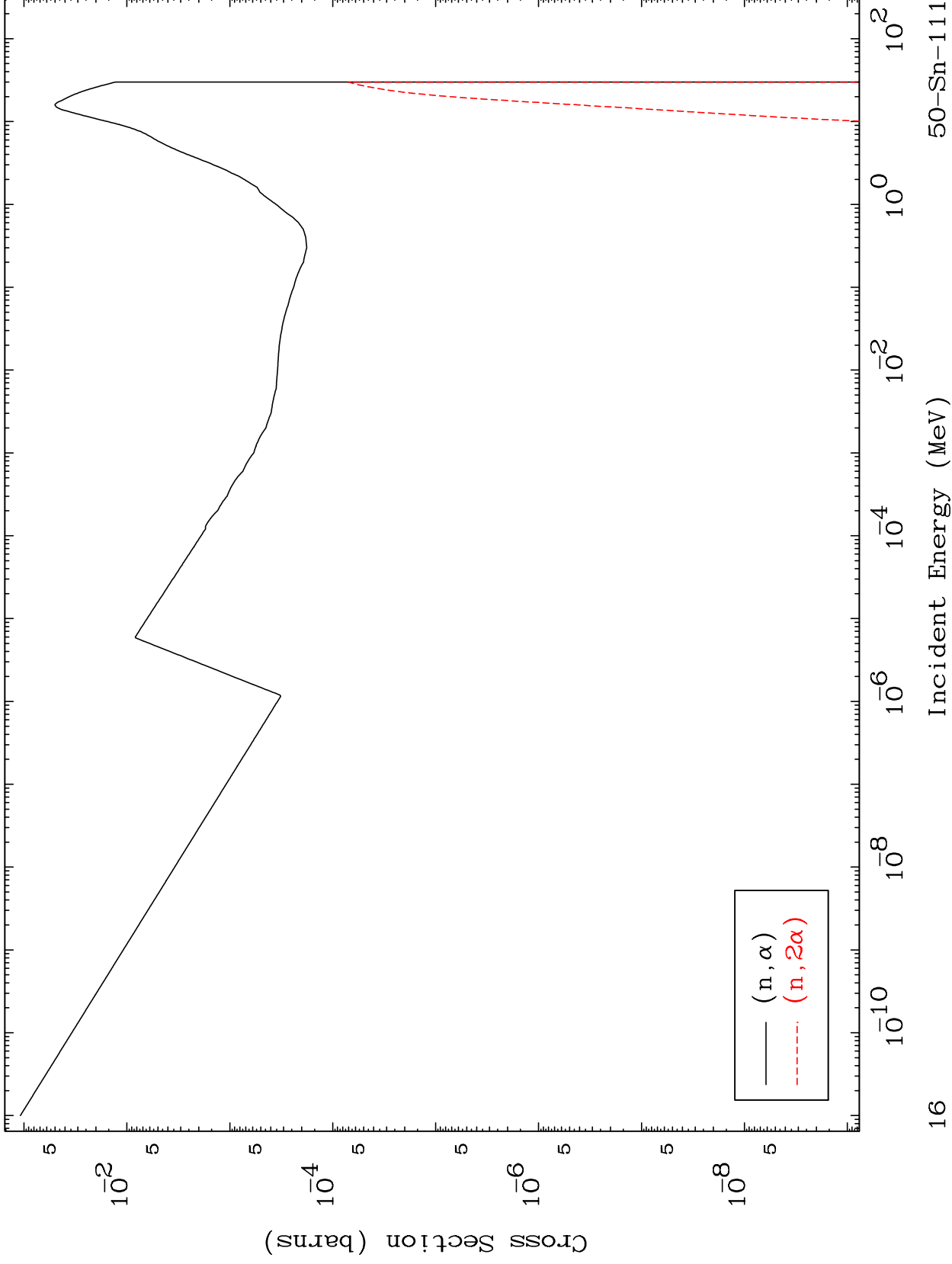
50-Sn-111



MAT 5022

(n,α) Levels  
293 Kelvin Cross Sections

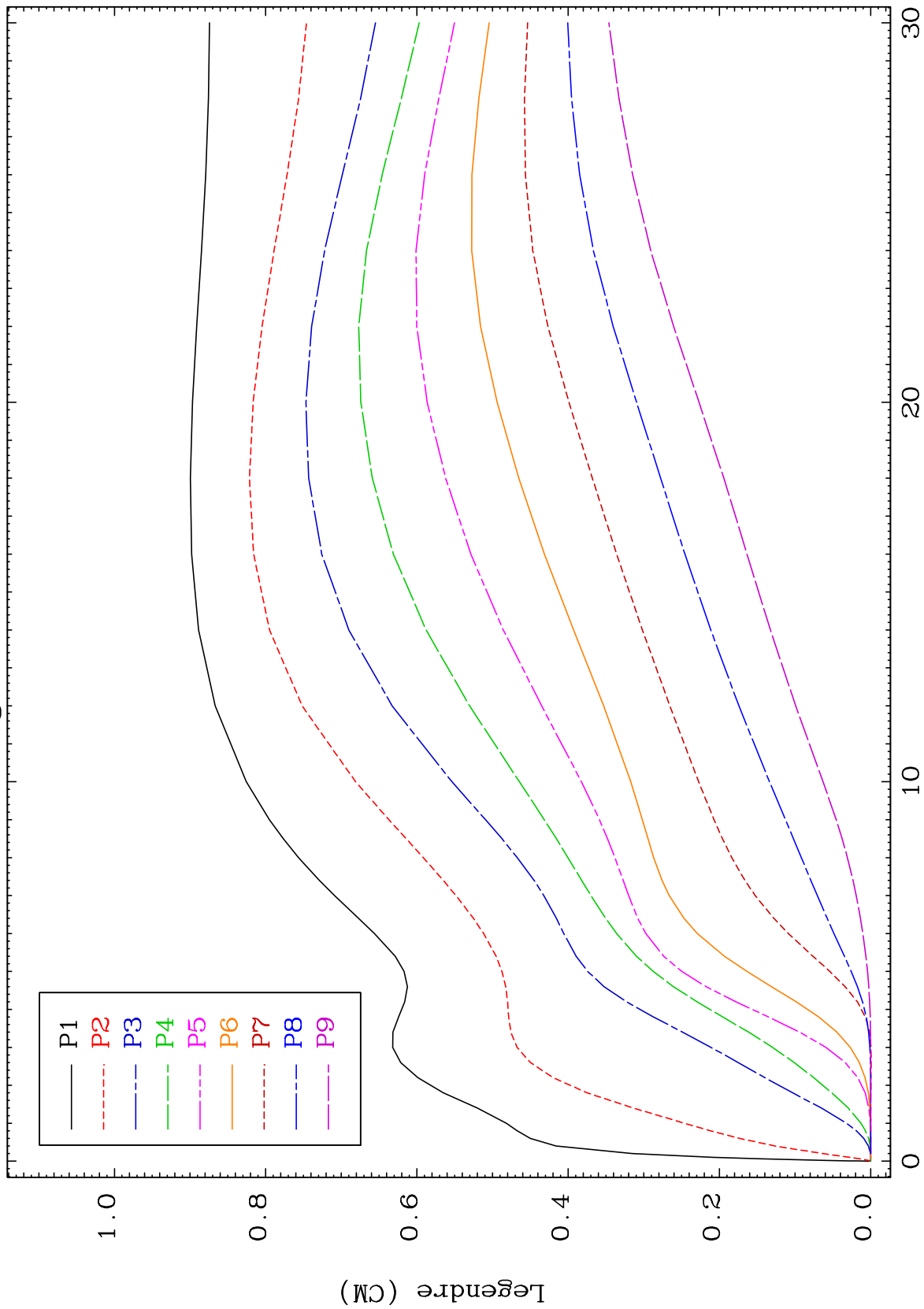
50-Sn-111



MAT 5022

Elastic Legendre Coefficients

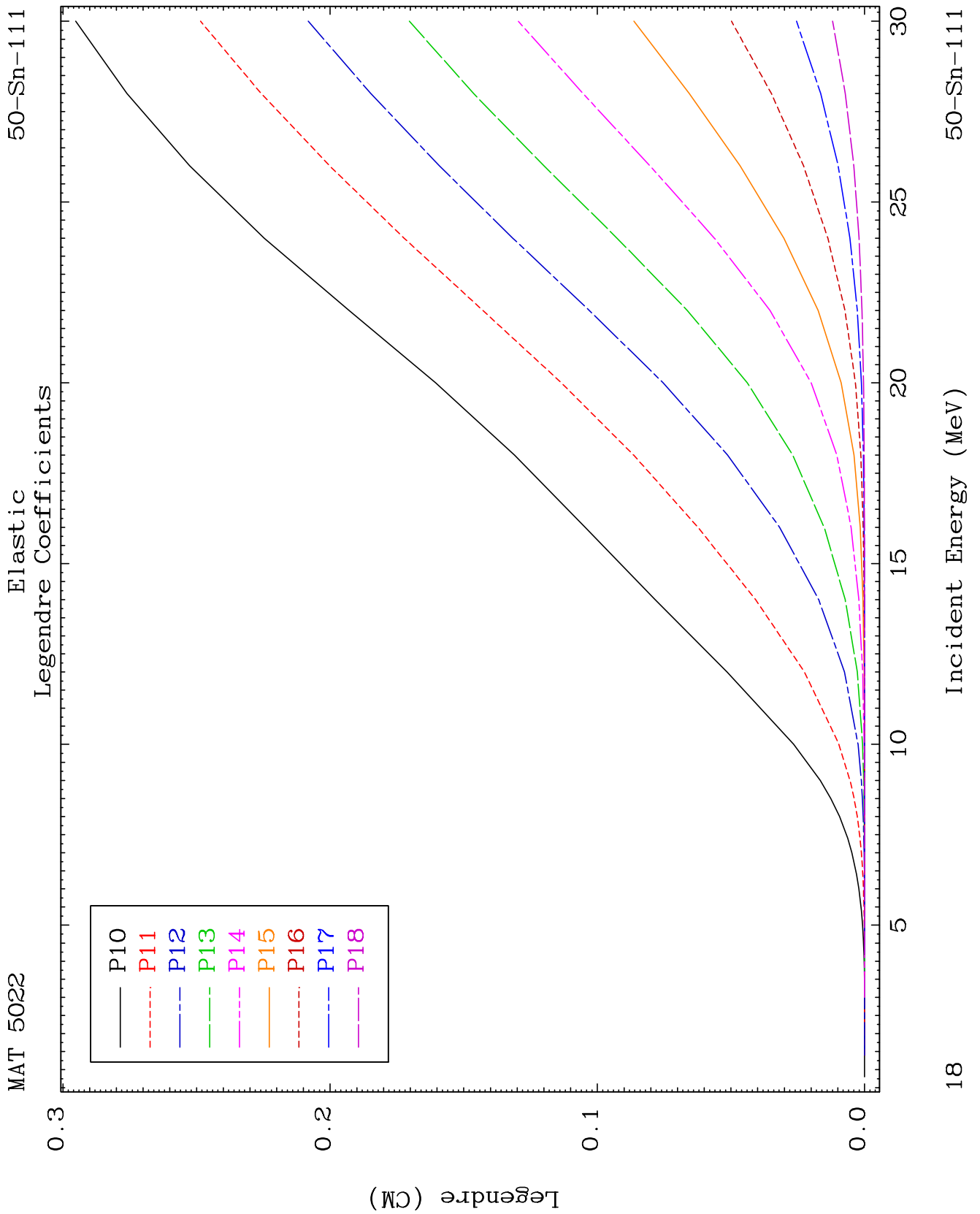
50-Sn-111



17

Incident Energy (MeV)

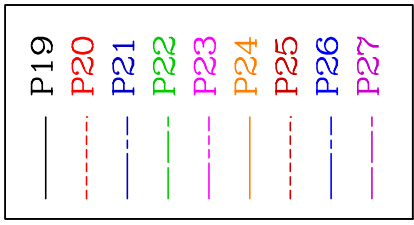
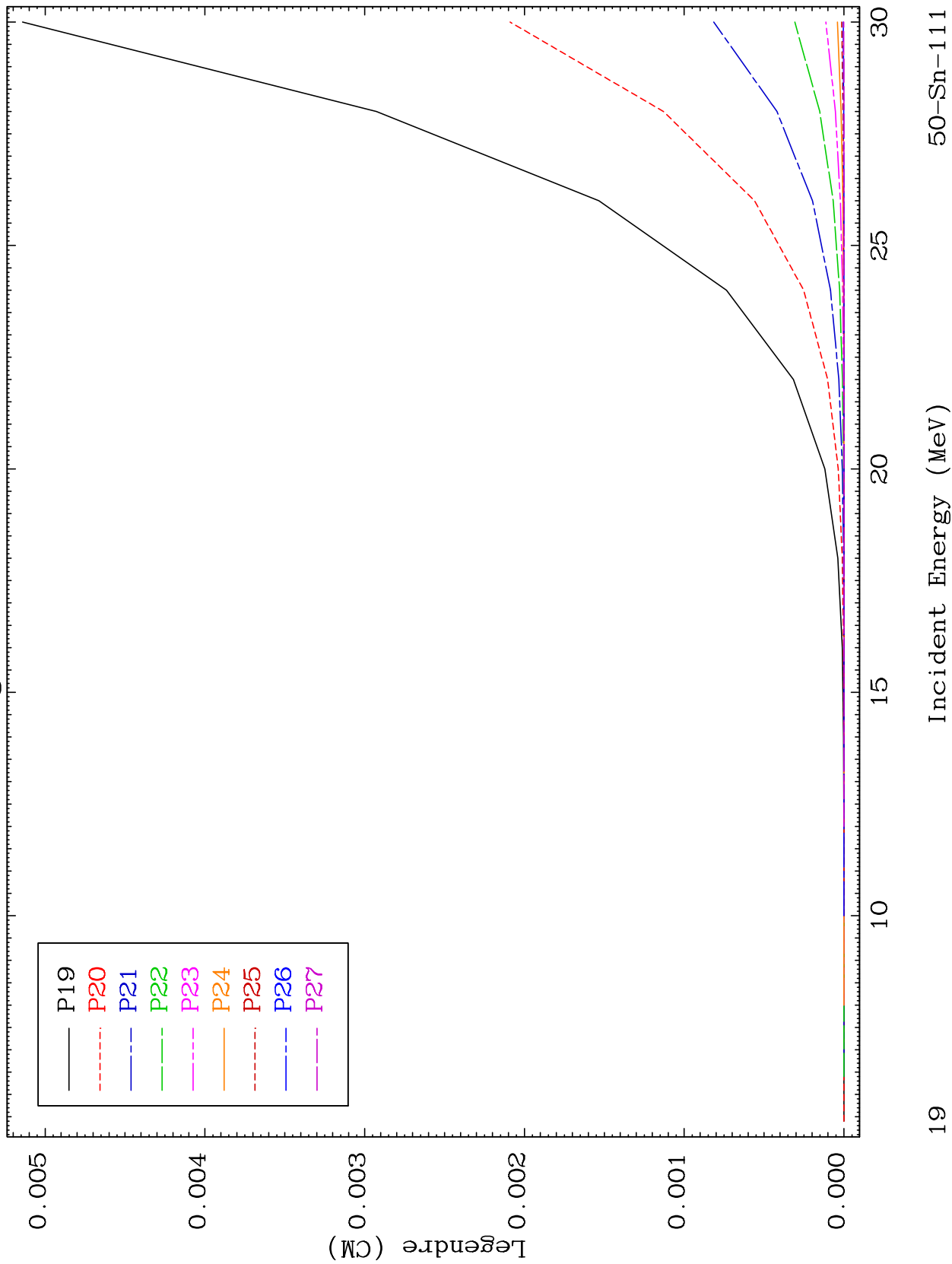
50-Sn-111



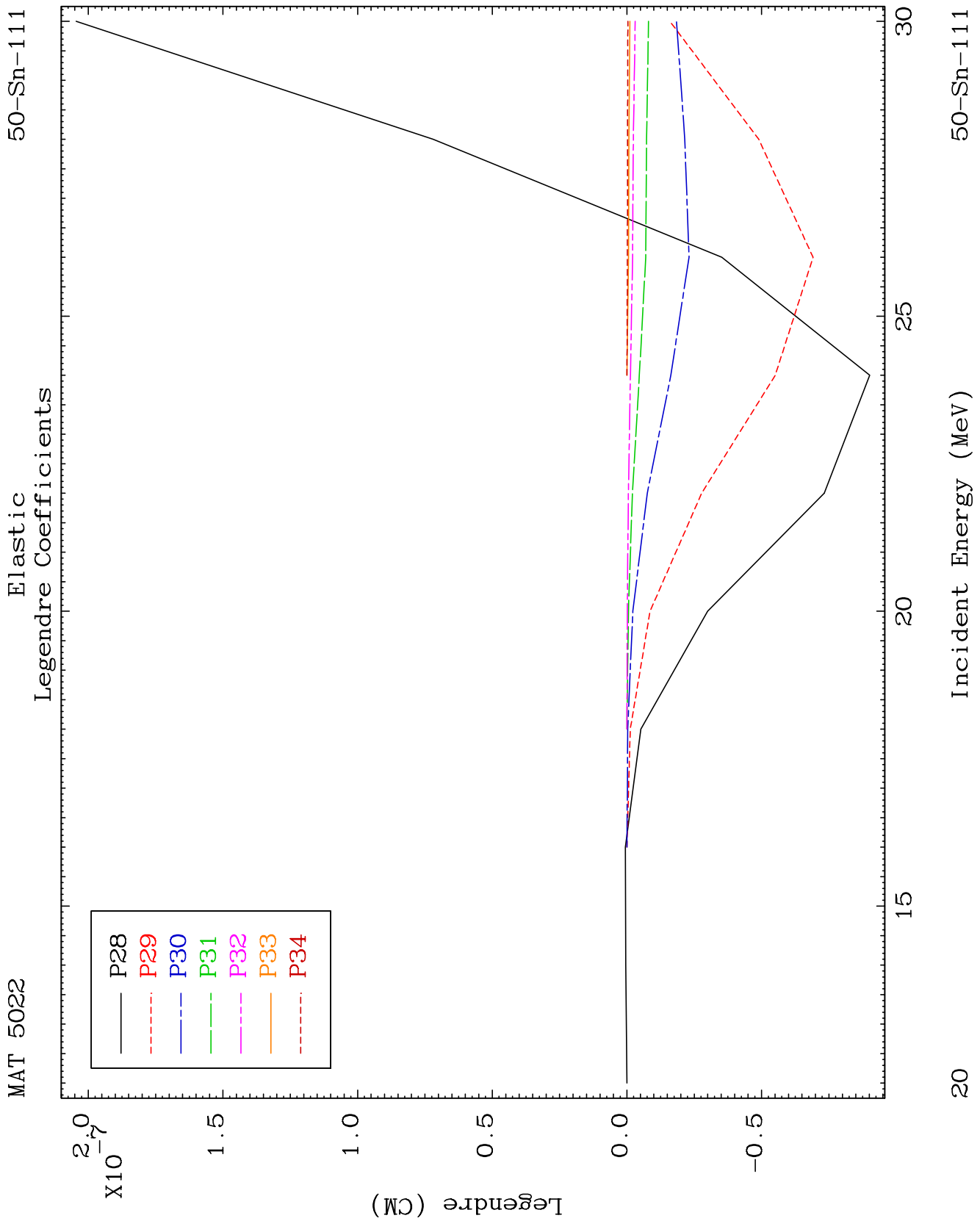
MAT 5022

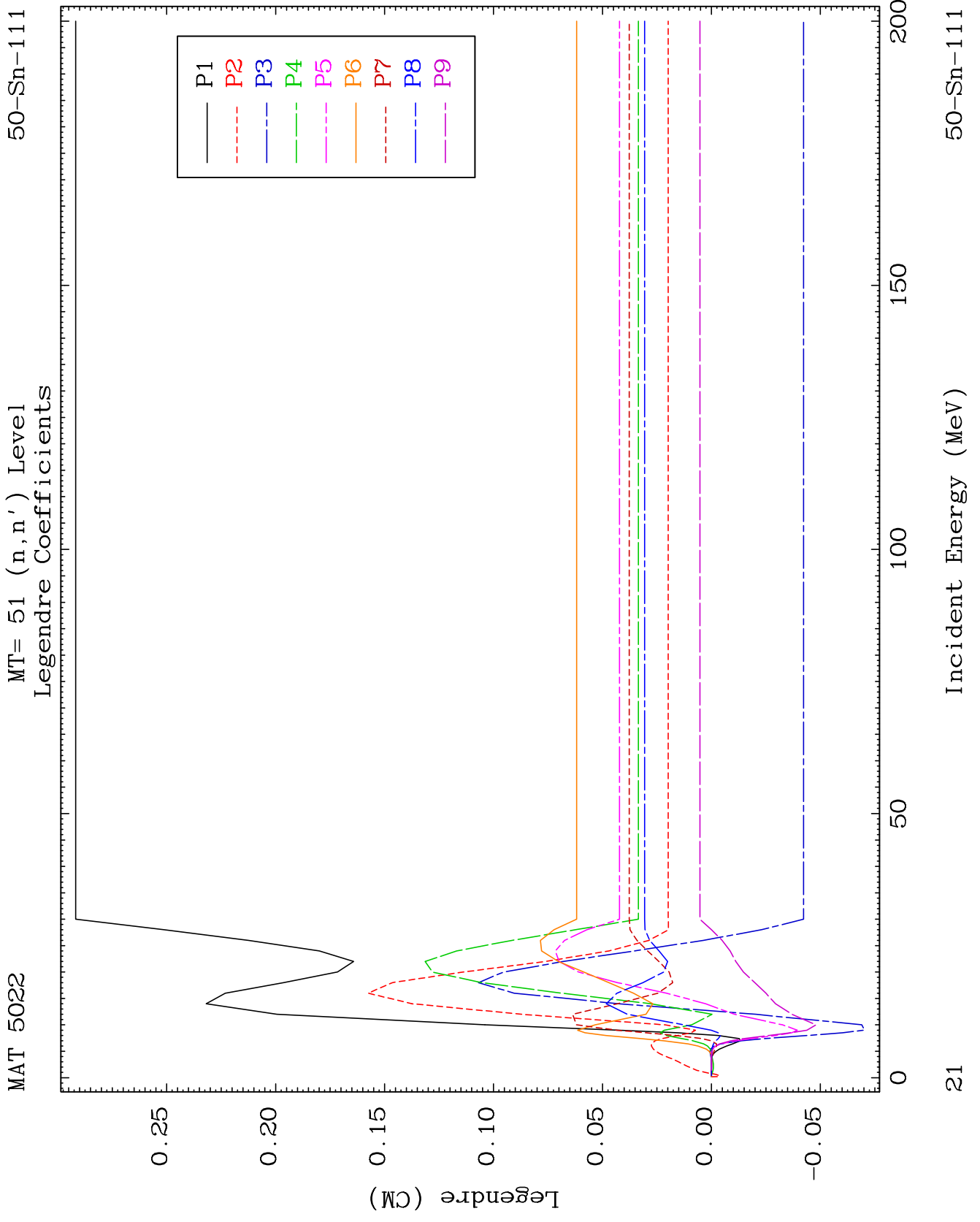
### Elastic Legendre Coefficients

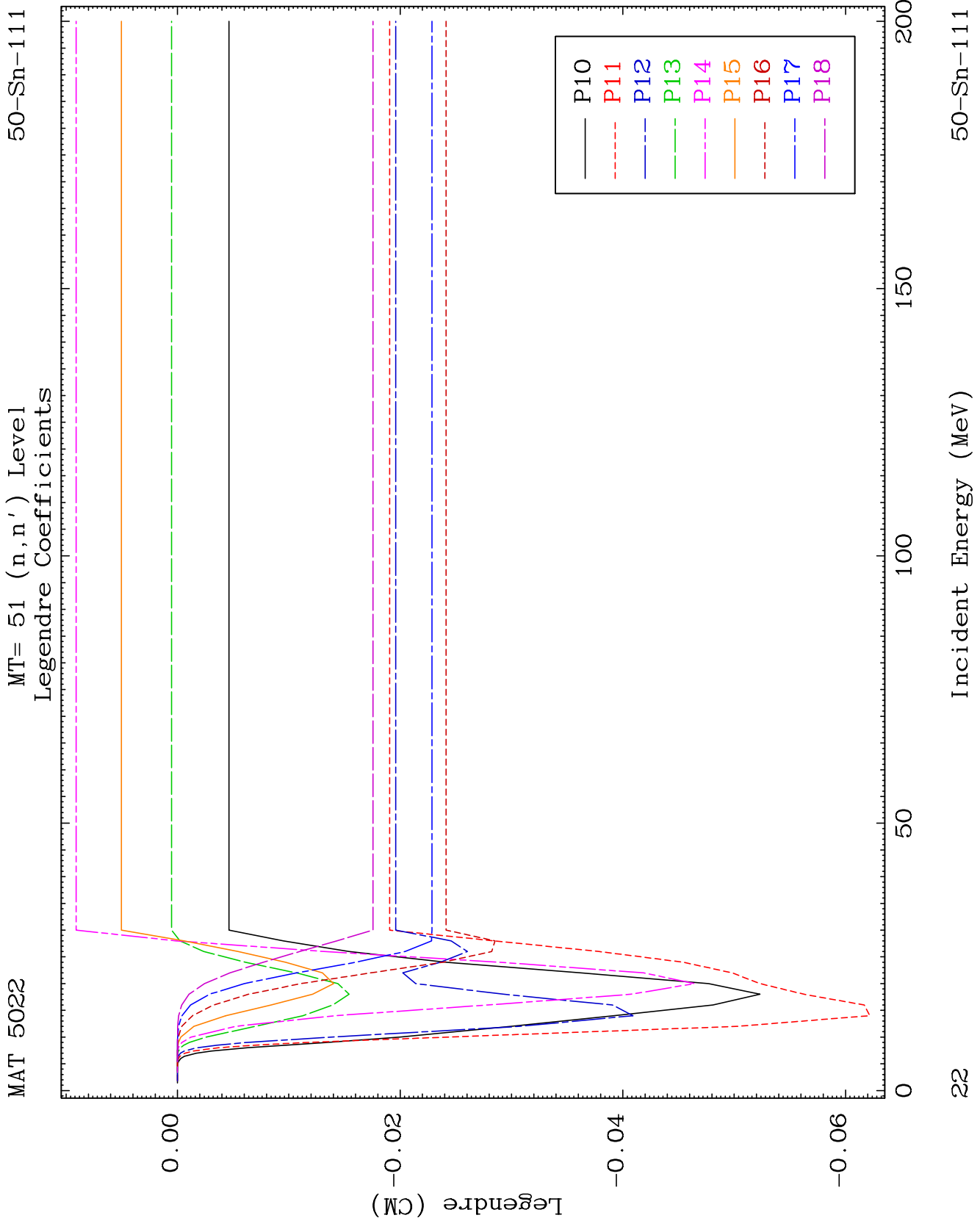
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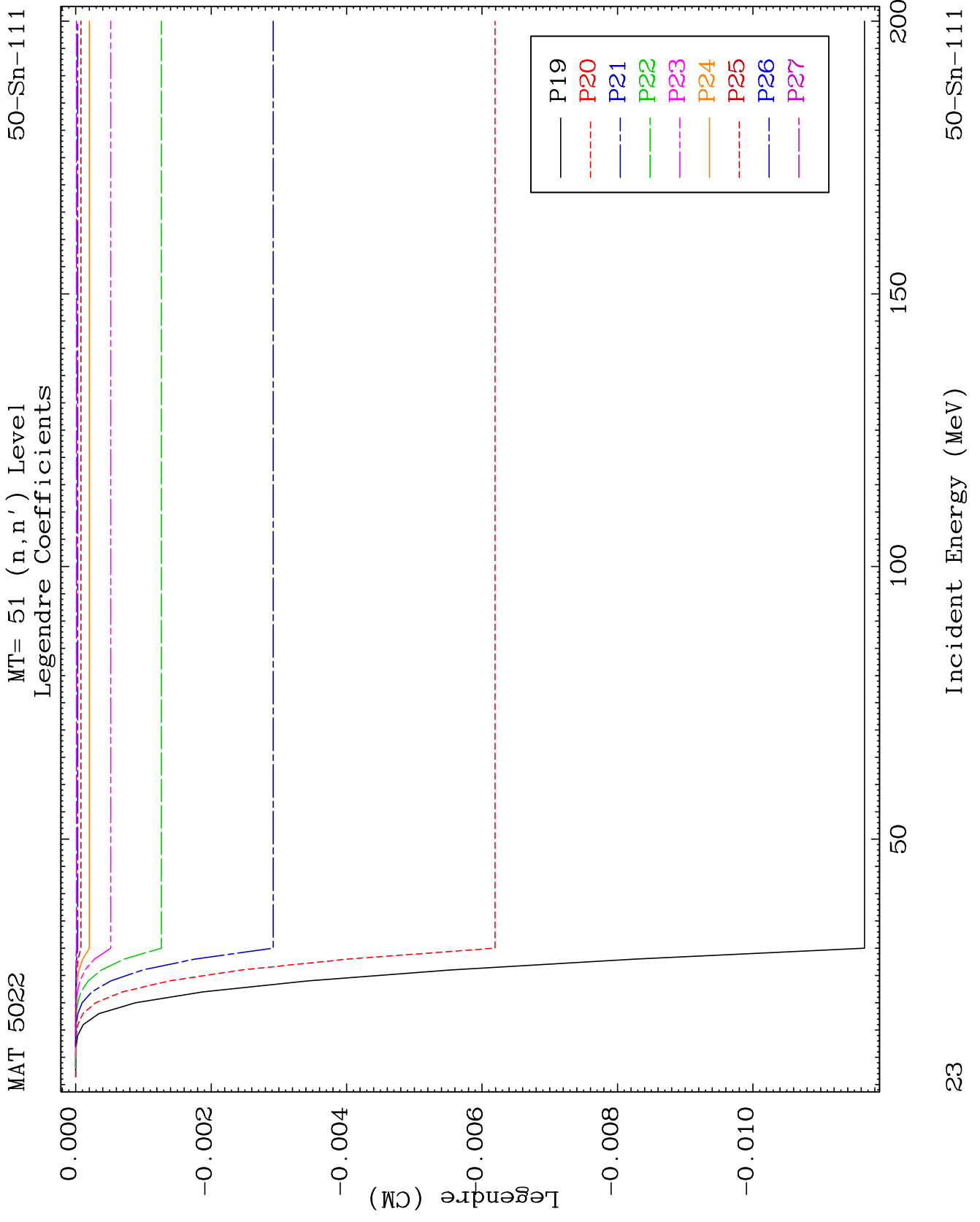


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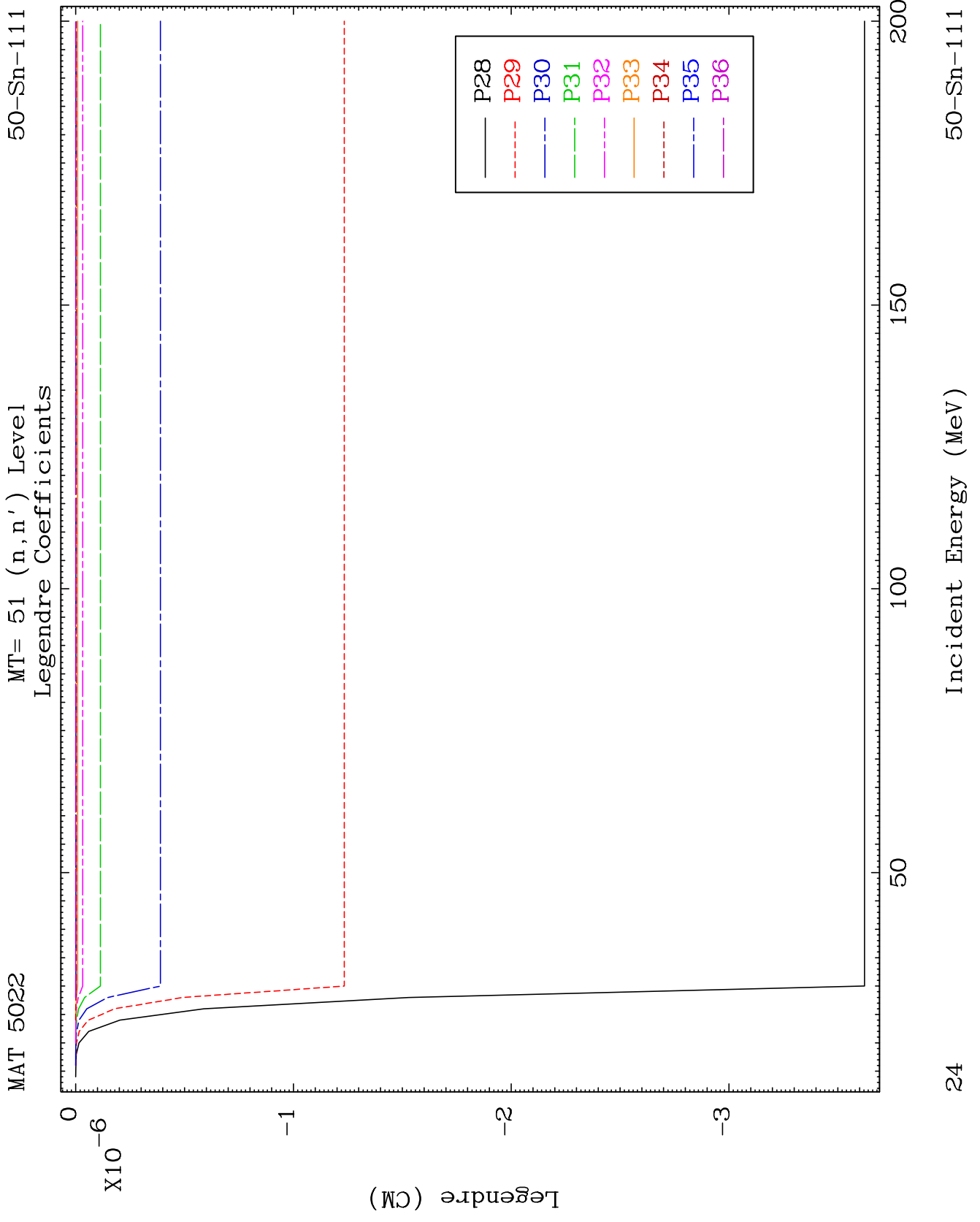










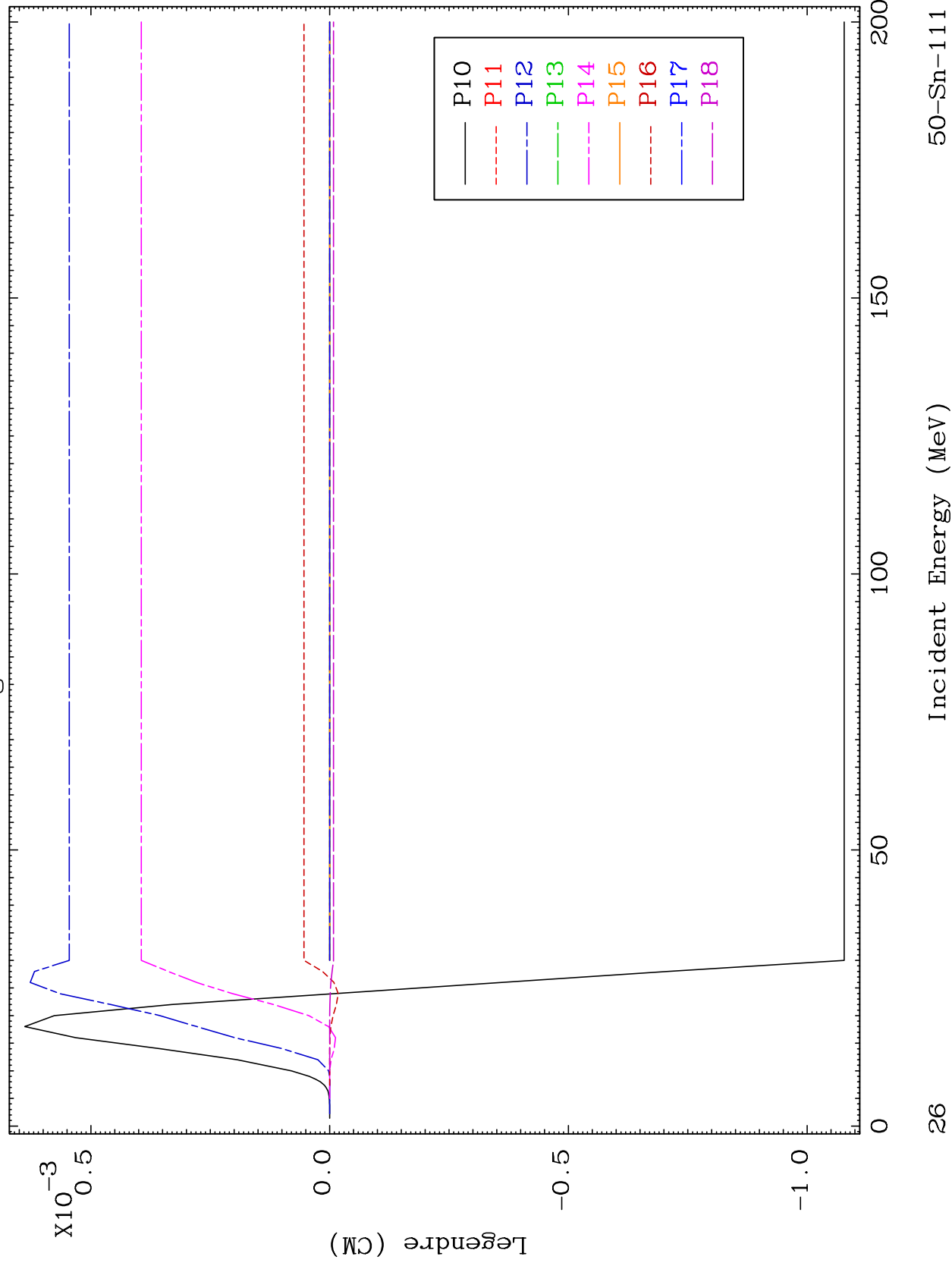




MAT 5022

MT= 52 (n,n') Level  
Legendre Coefficients

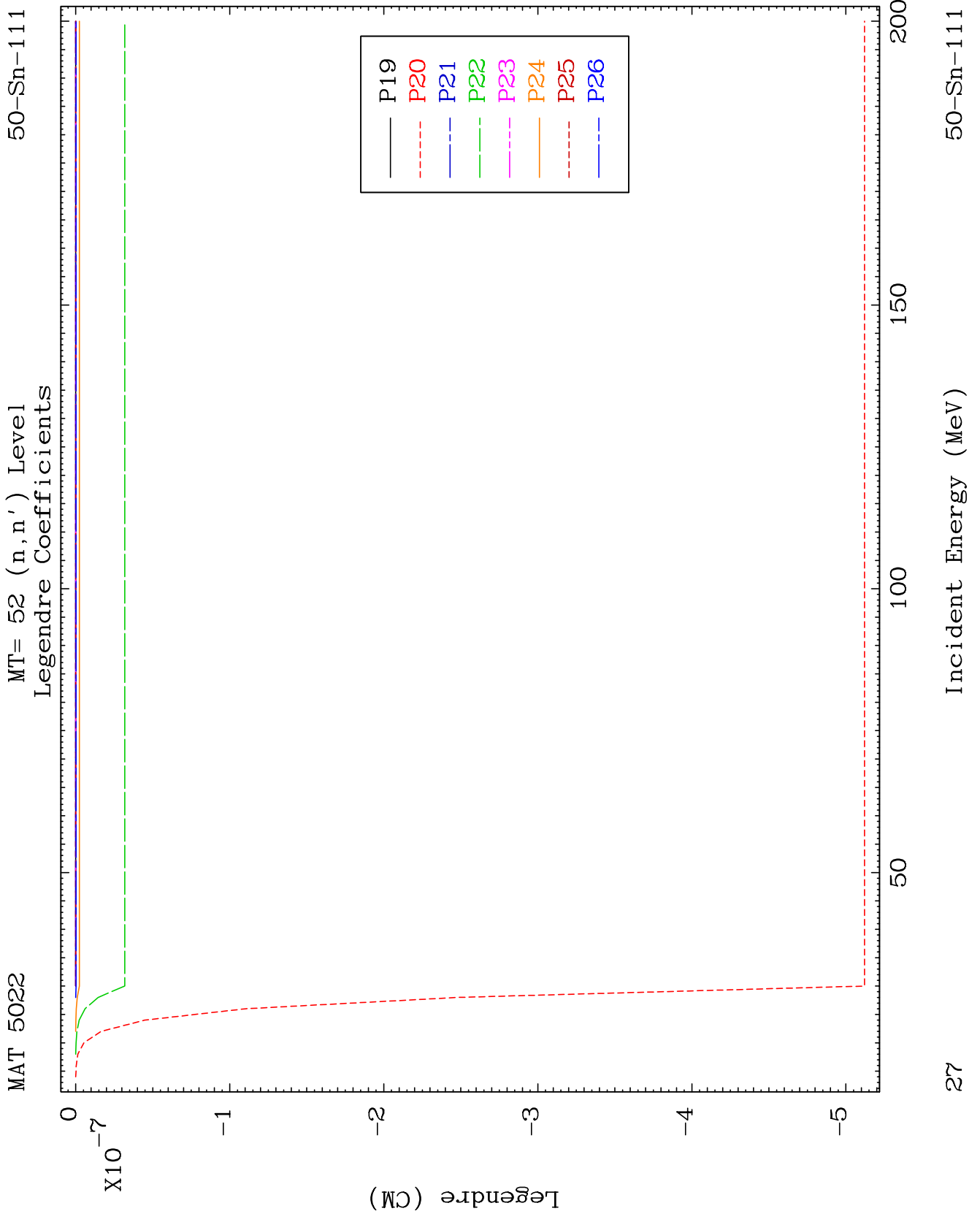
50-Sn-111



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Incident Energy (MeV)

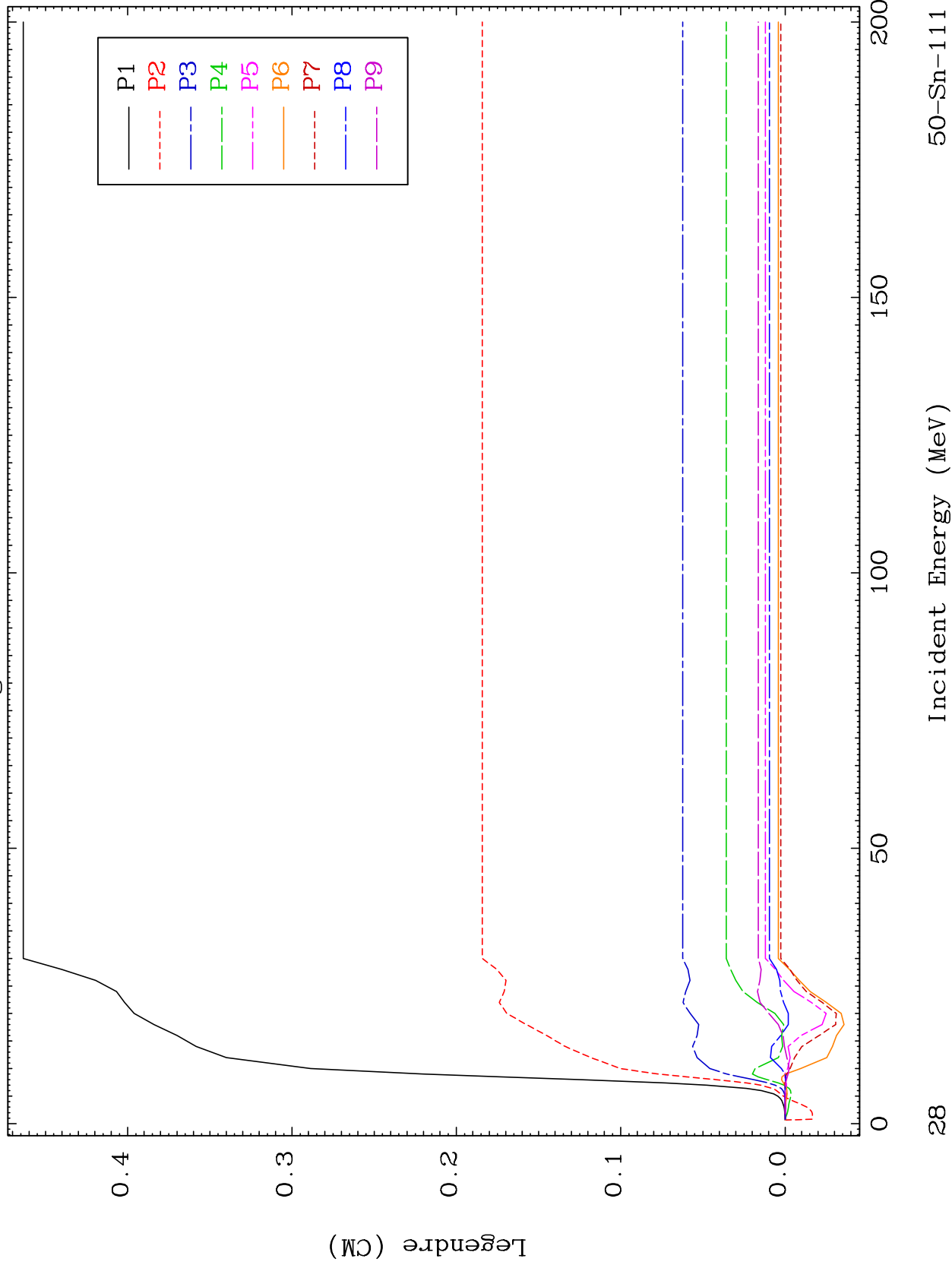
50-Sn-111



MAT 5022

MT= 53 (n,n') Level  
Legendre Coefficients

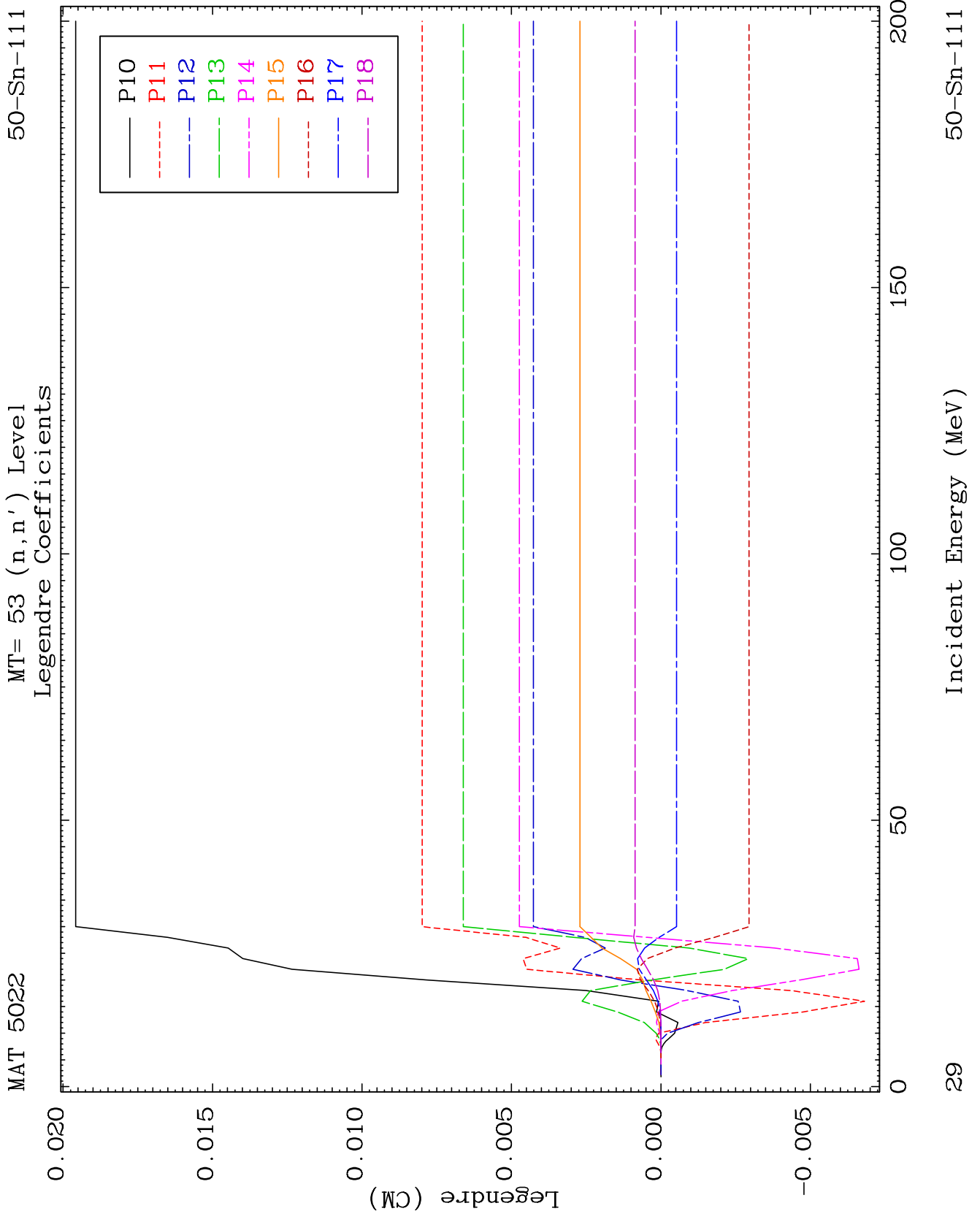
50-Sn-111



50-Sn-111

Incident Energy (MeV)

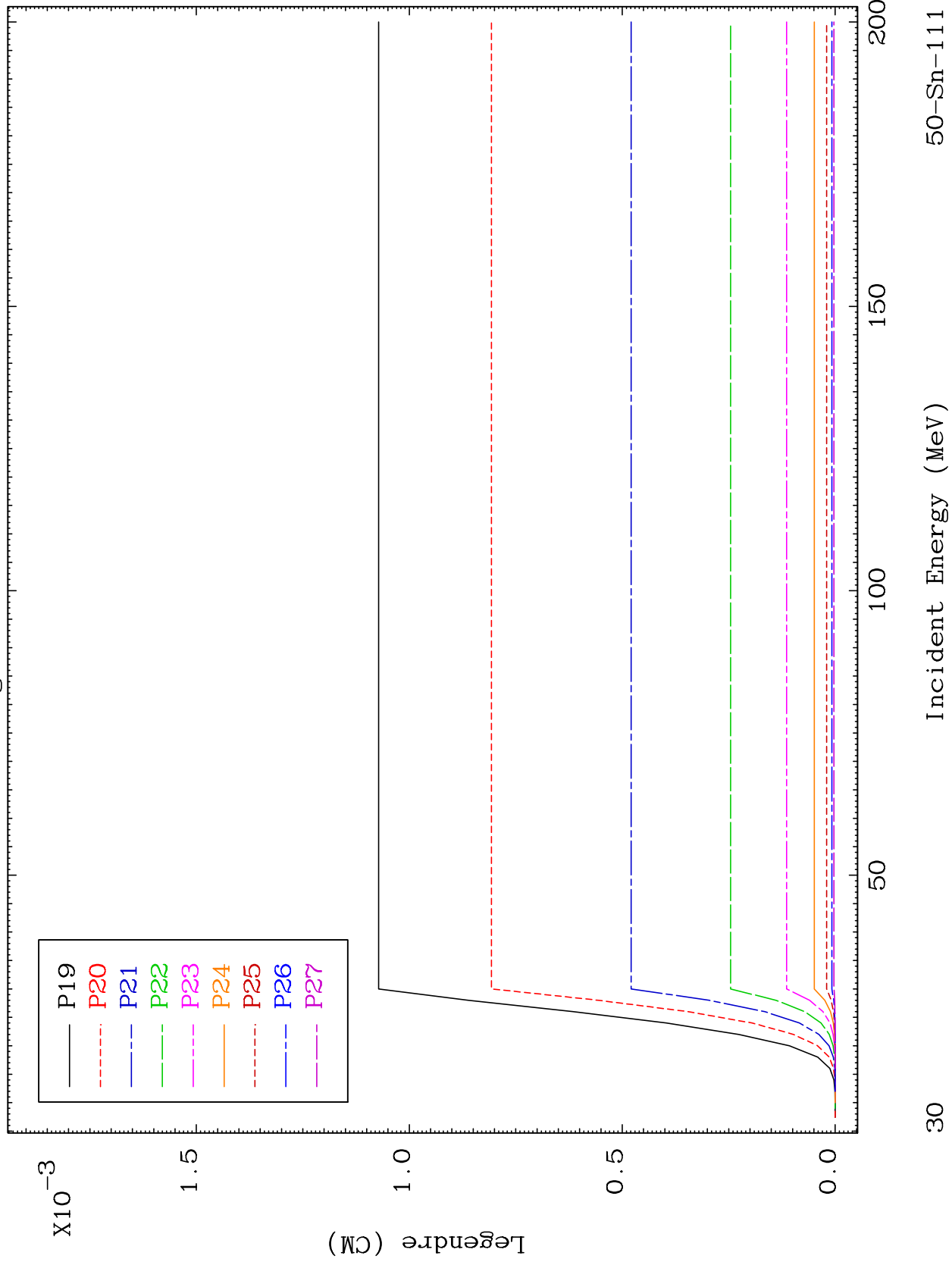
28

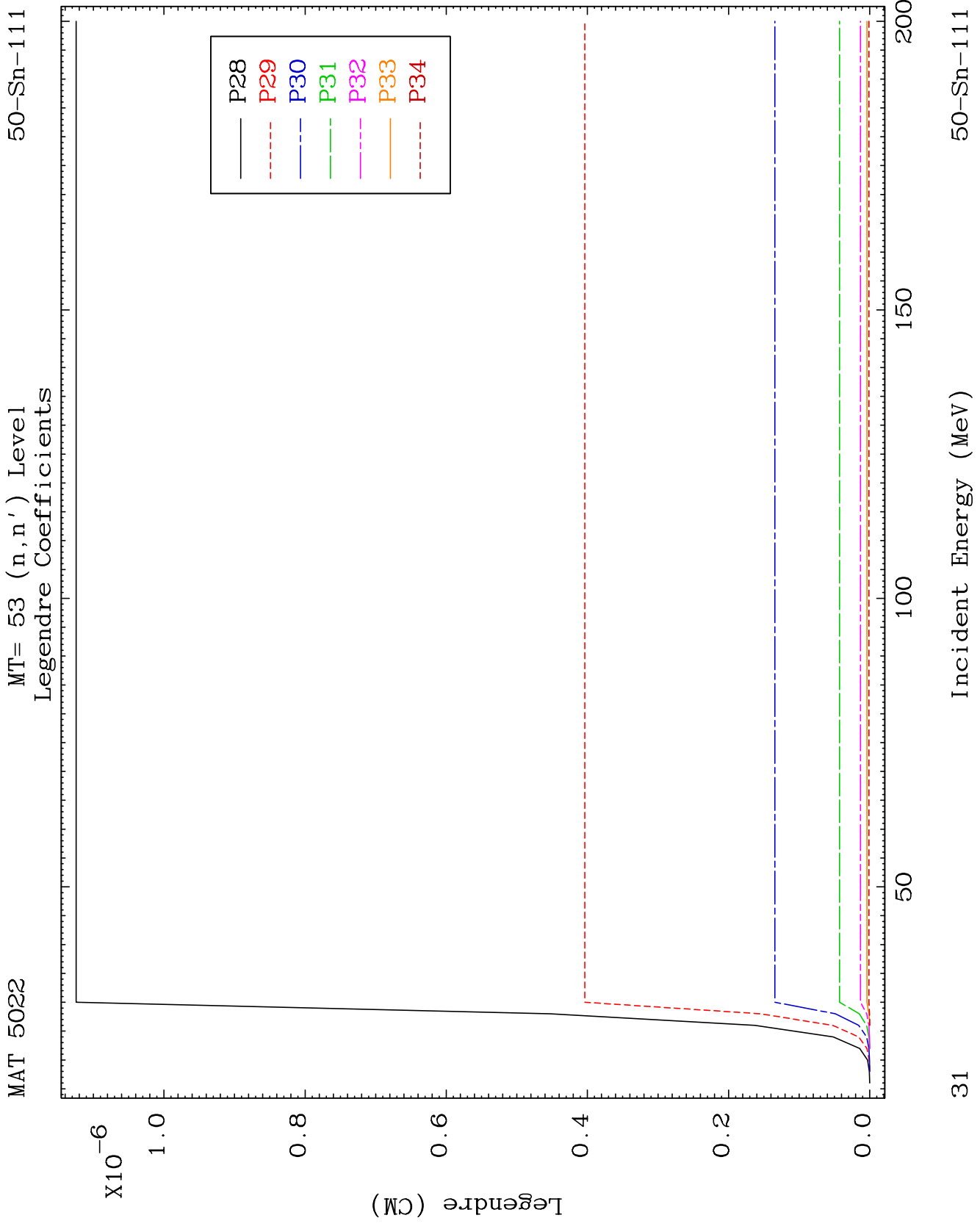


MAT 5022

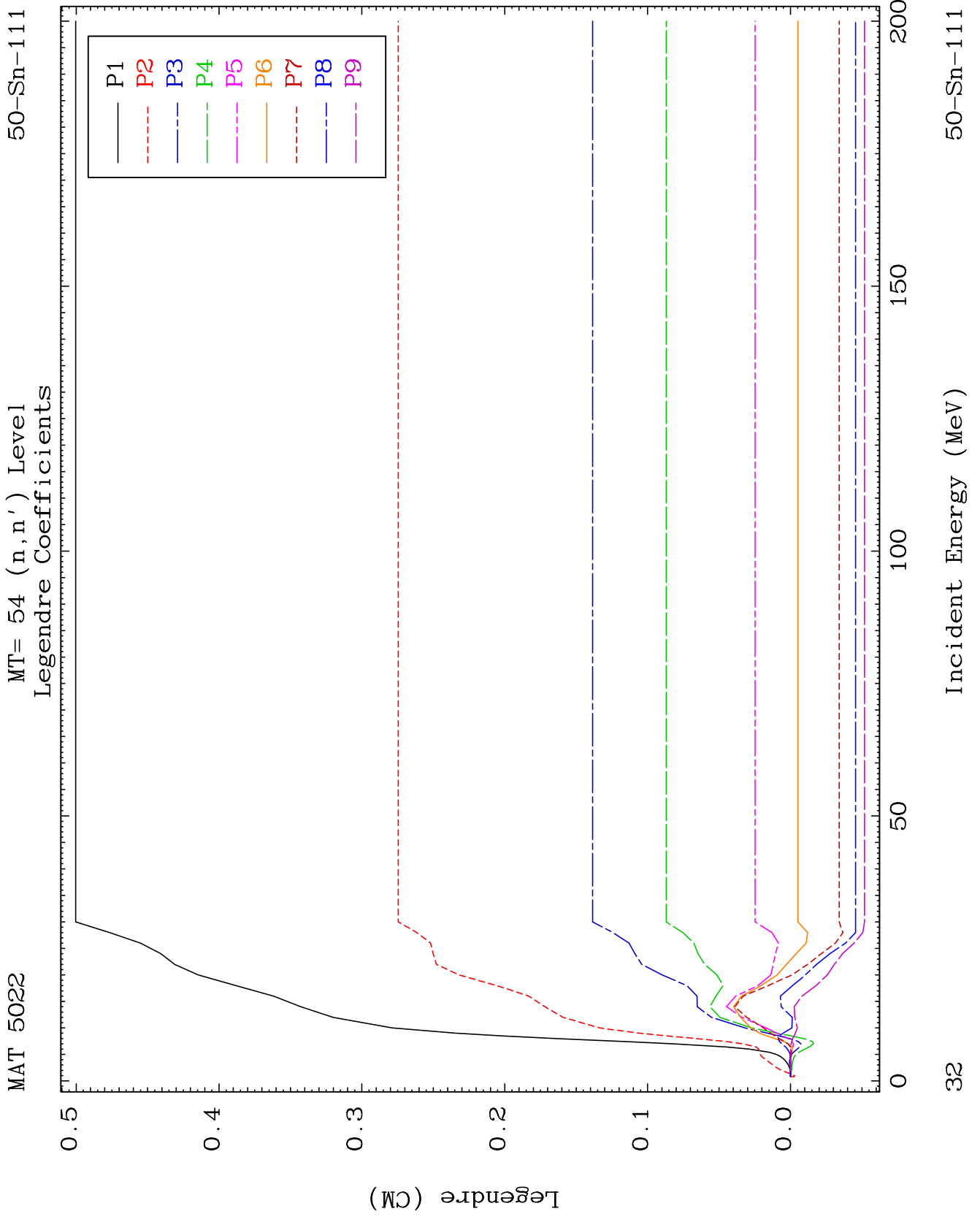
MT= 53 (n,n') Level  
Legendre Coefficients

50-Sn-111





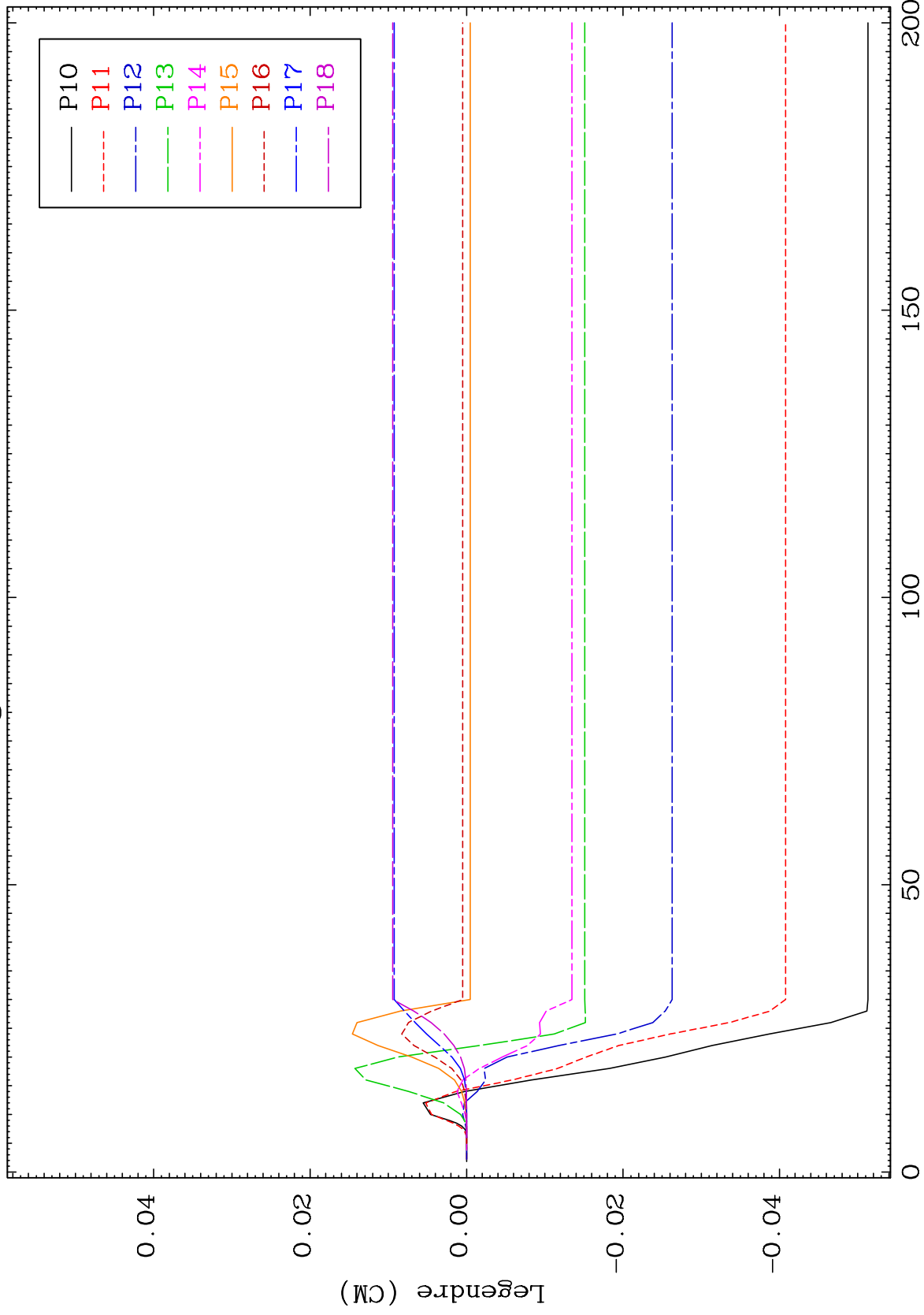




MAT 5022

MT= 54 (n,n') Level  
Legendre Coefficients

50-Sn-111

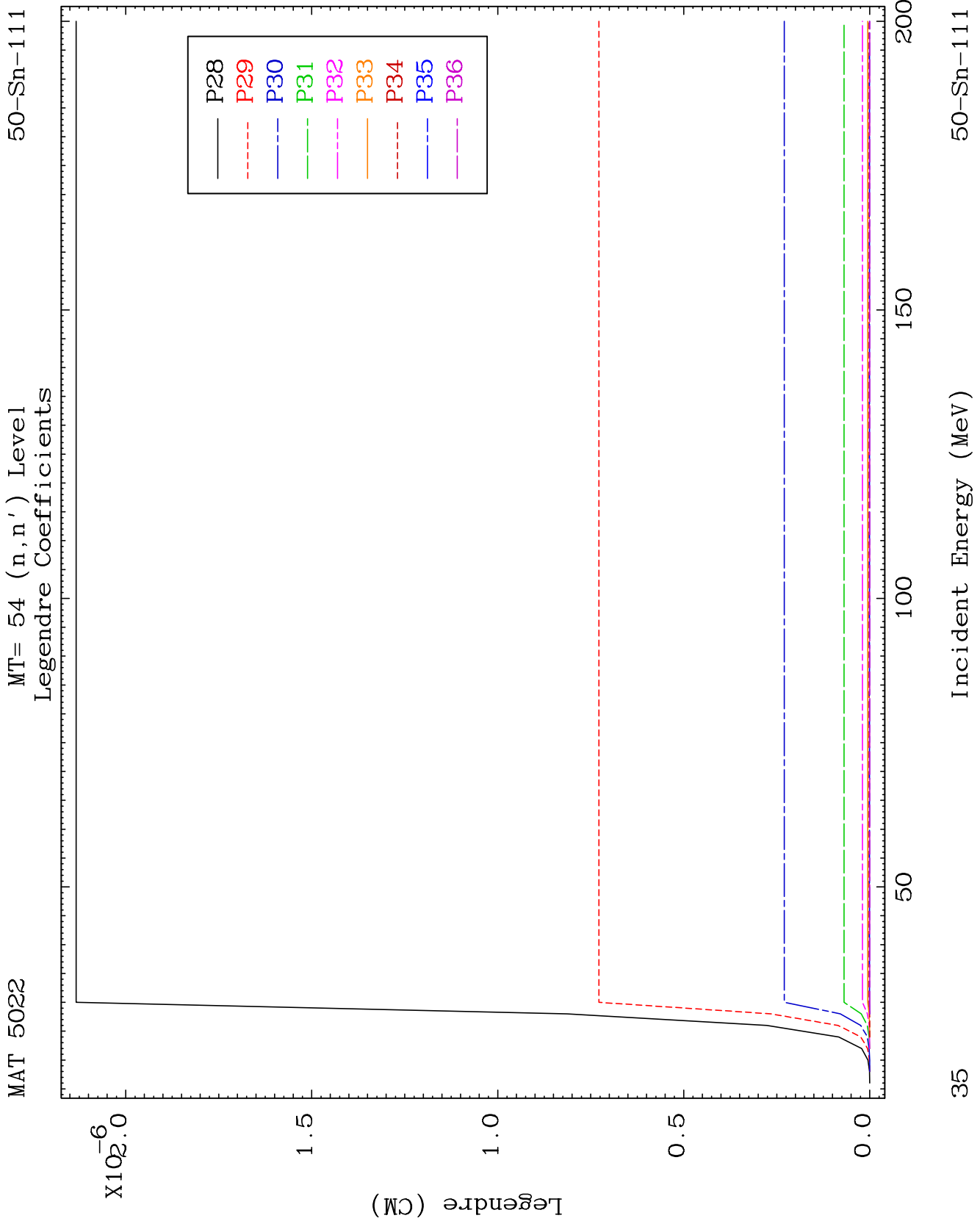


33

Incident Energy (MeV)

50-Sn-111



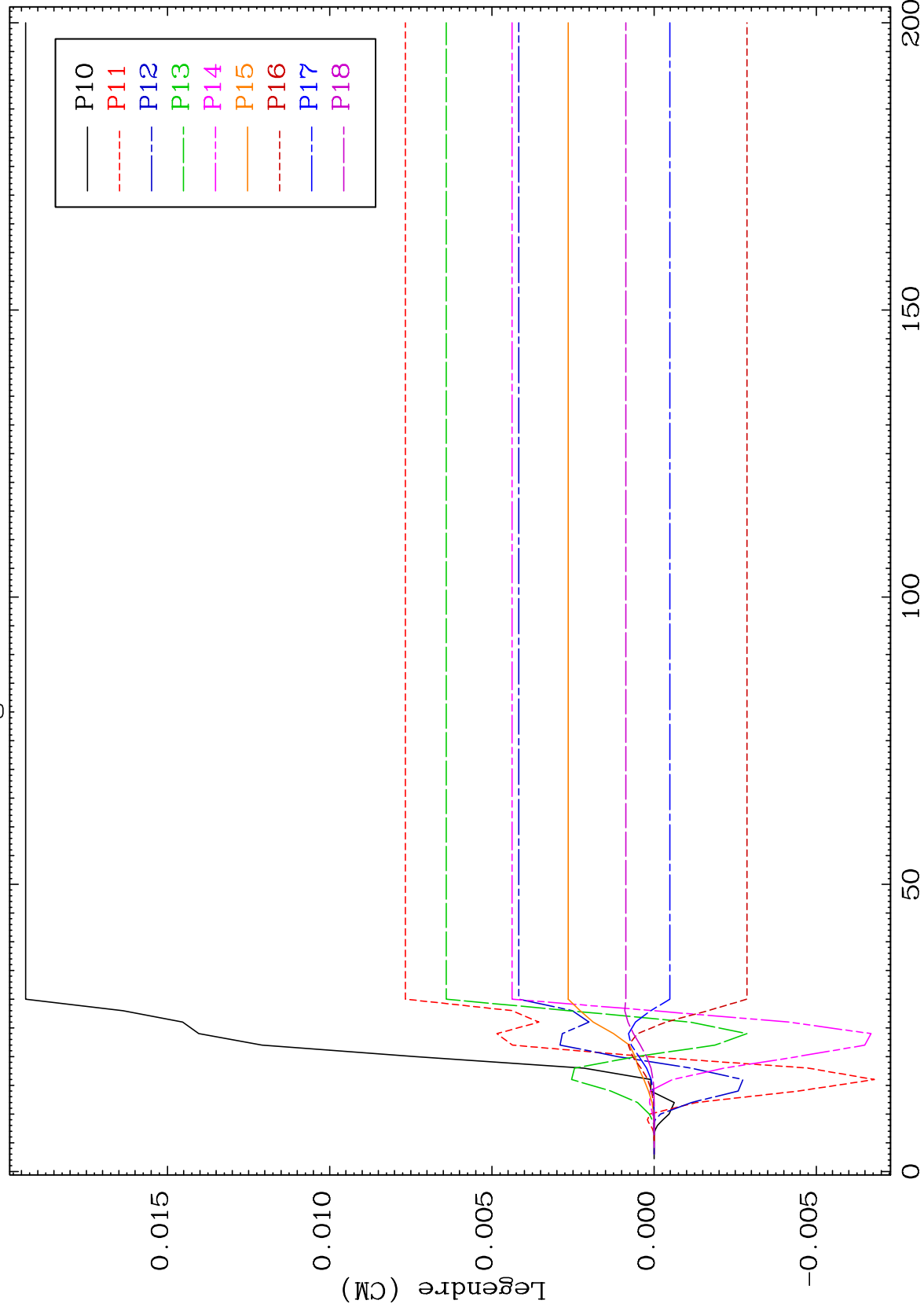




MAT 5022

MT= 55 (n,n') Level  
Legendre Coefficients

50-Sn-111



37

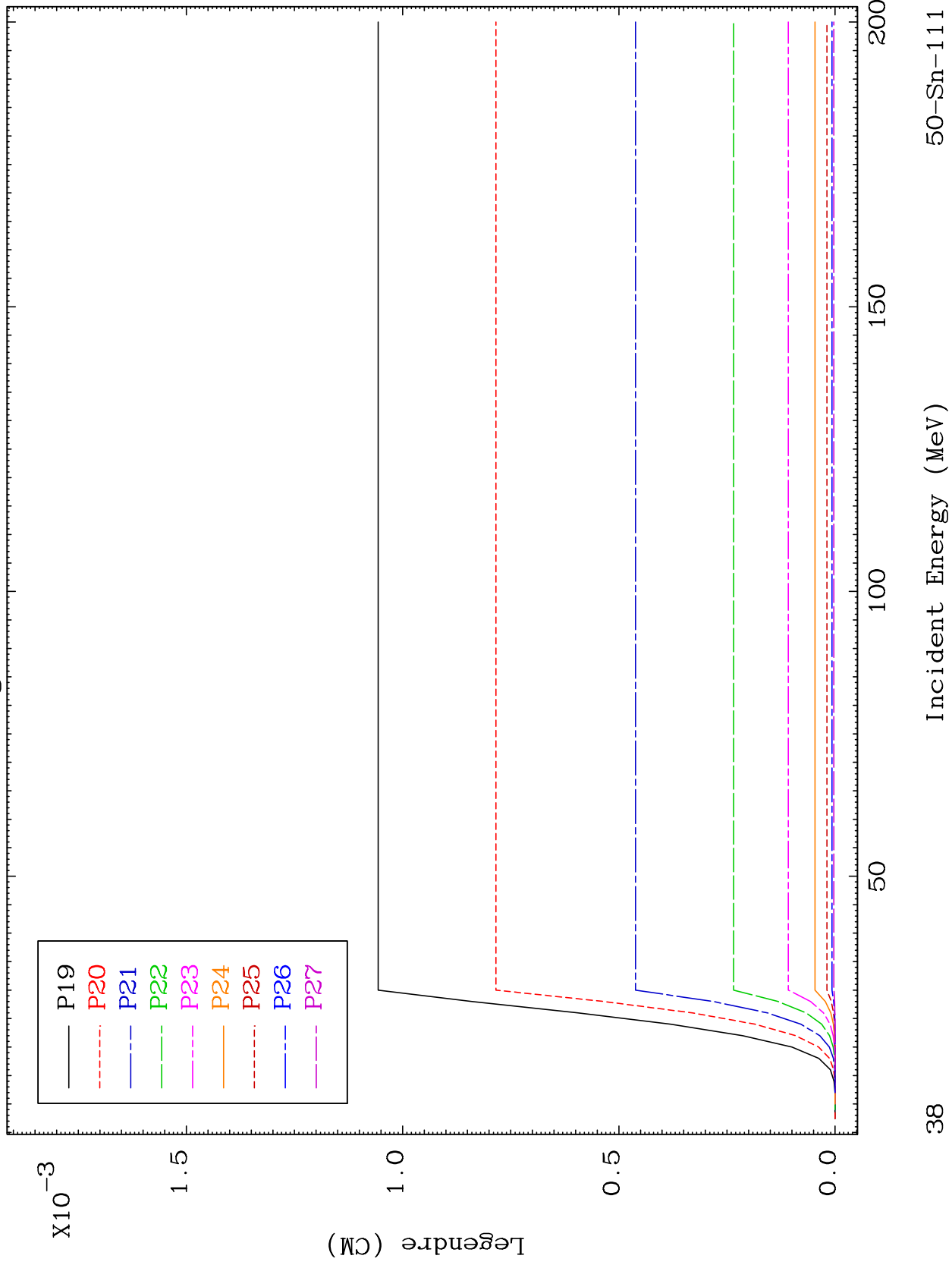
Incident Energy (MeV)

50-Sn-111

MAT 5022

MT= 55 (n,n') Level  
Legendre Coefficients

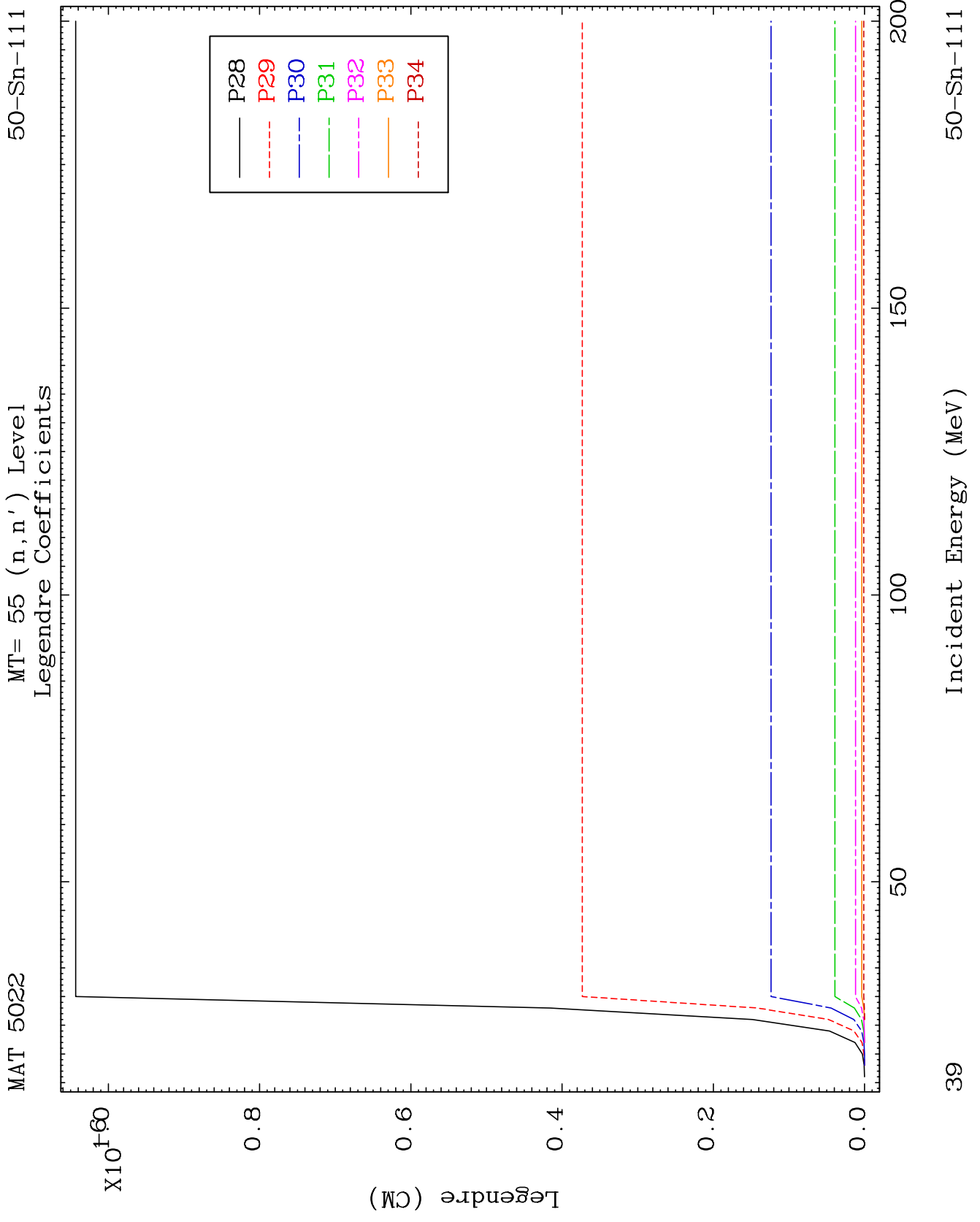
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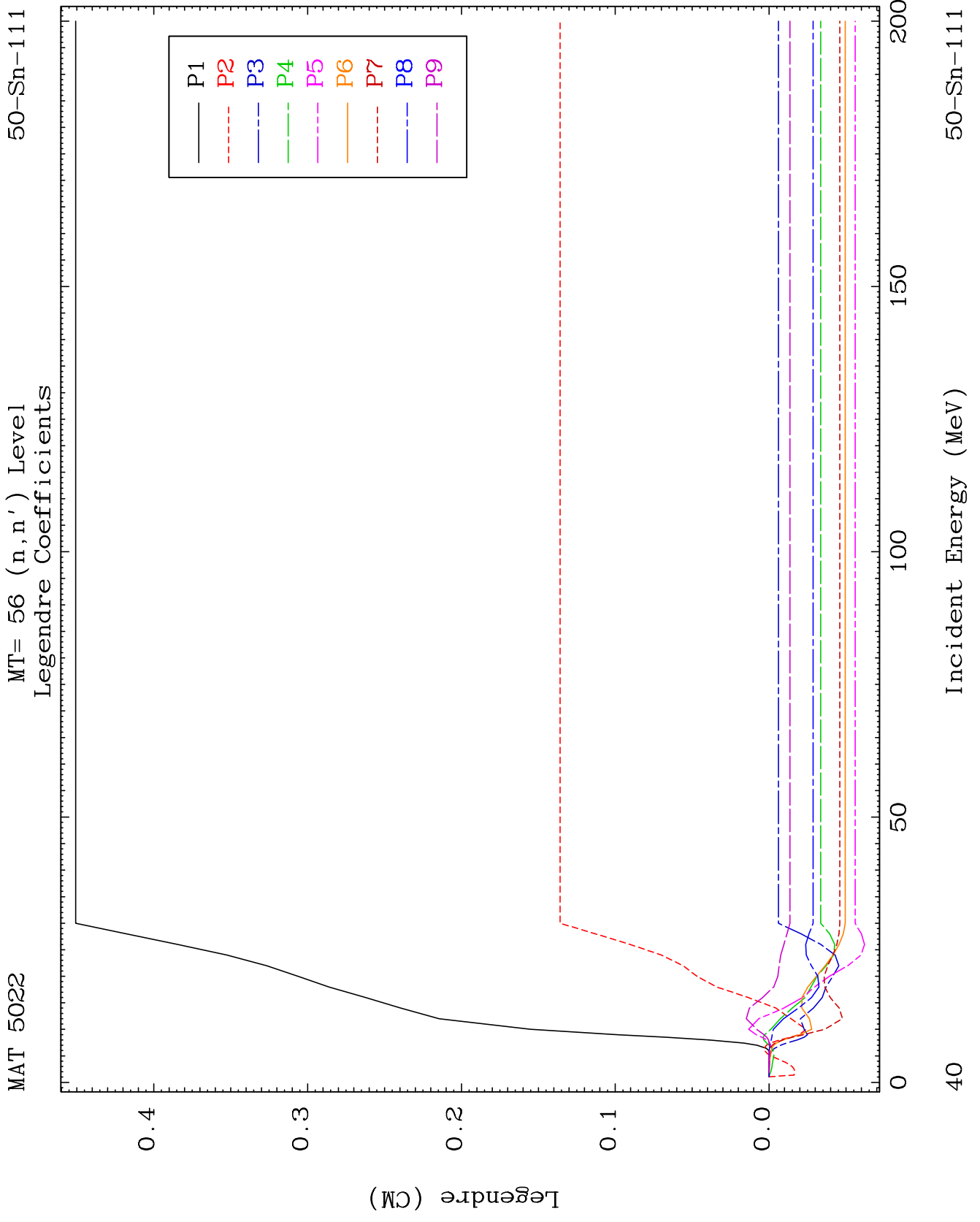
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Incident Energy (MeV)

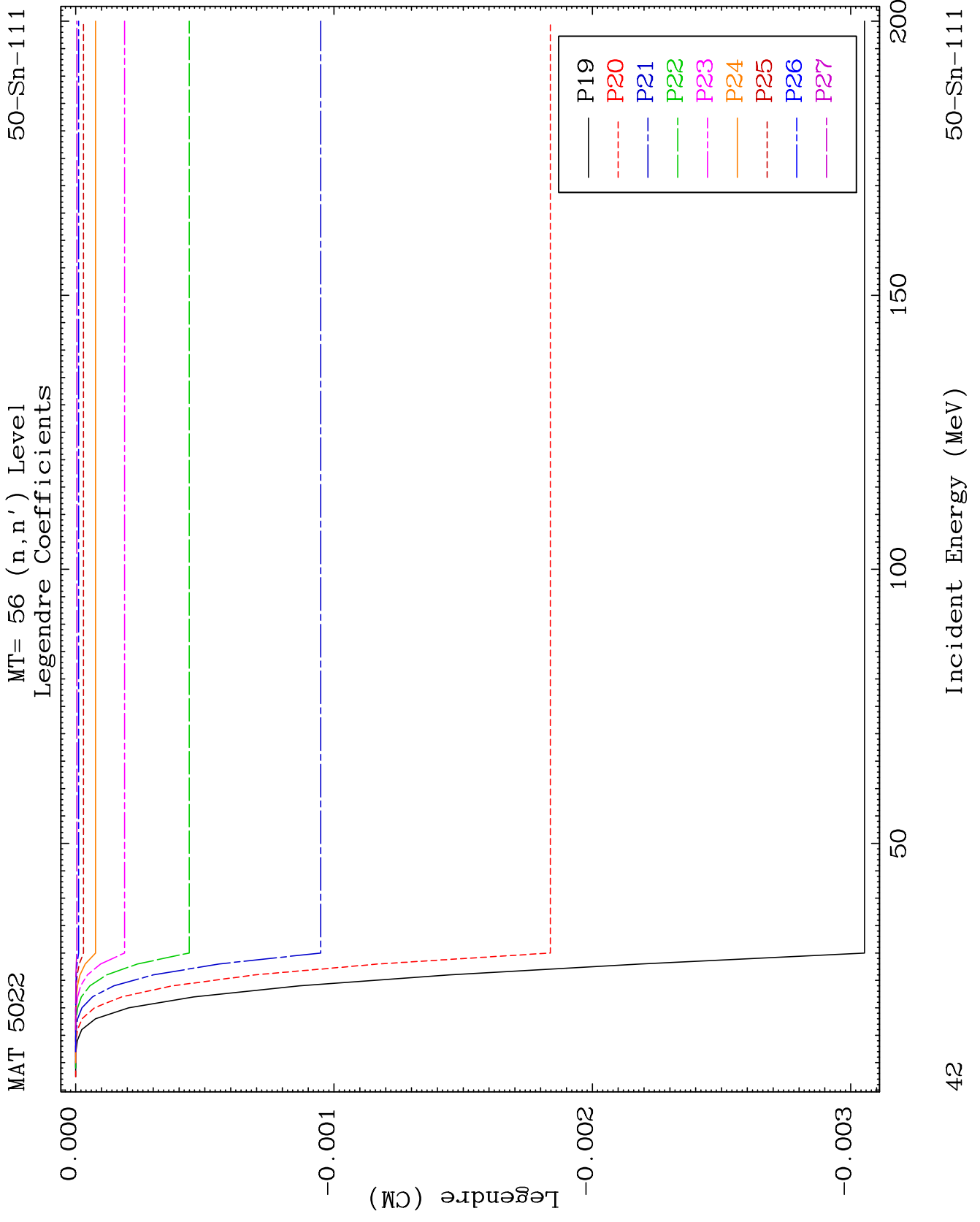
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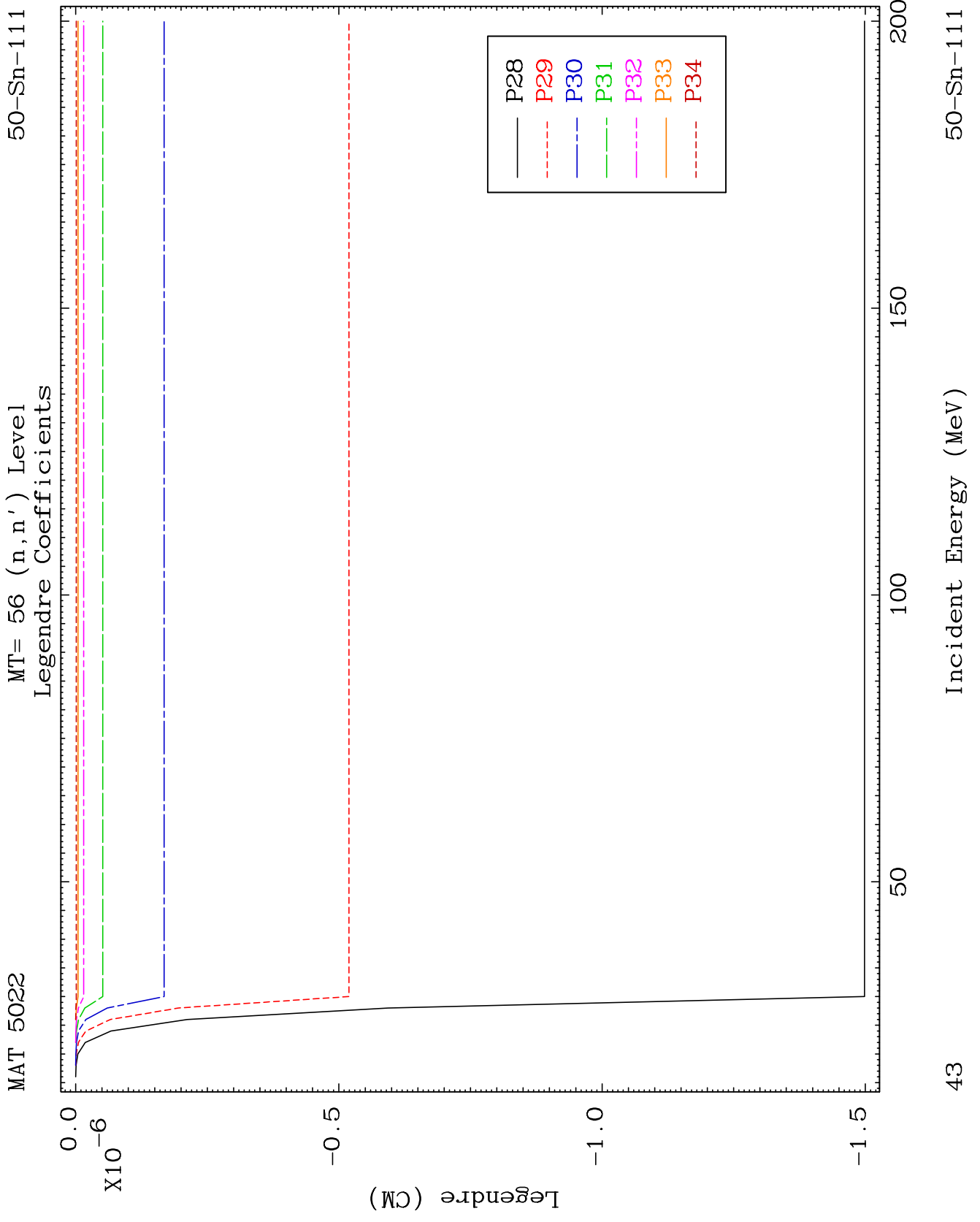


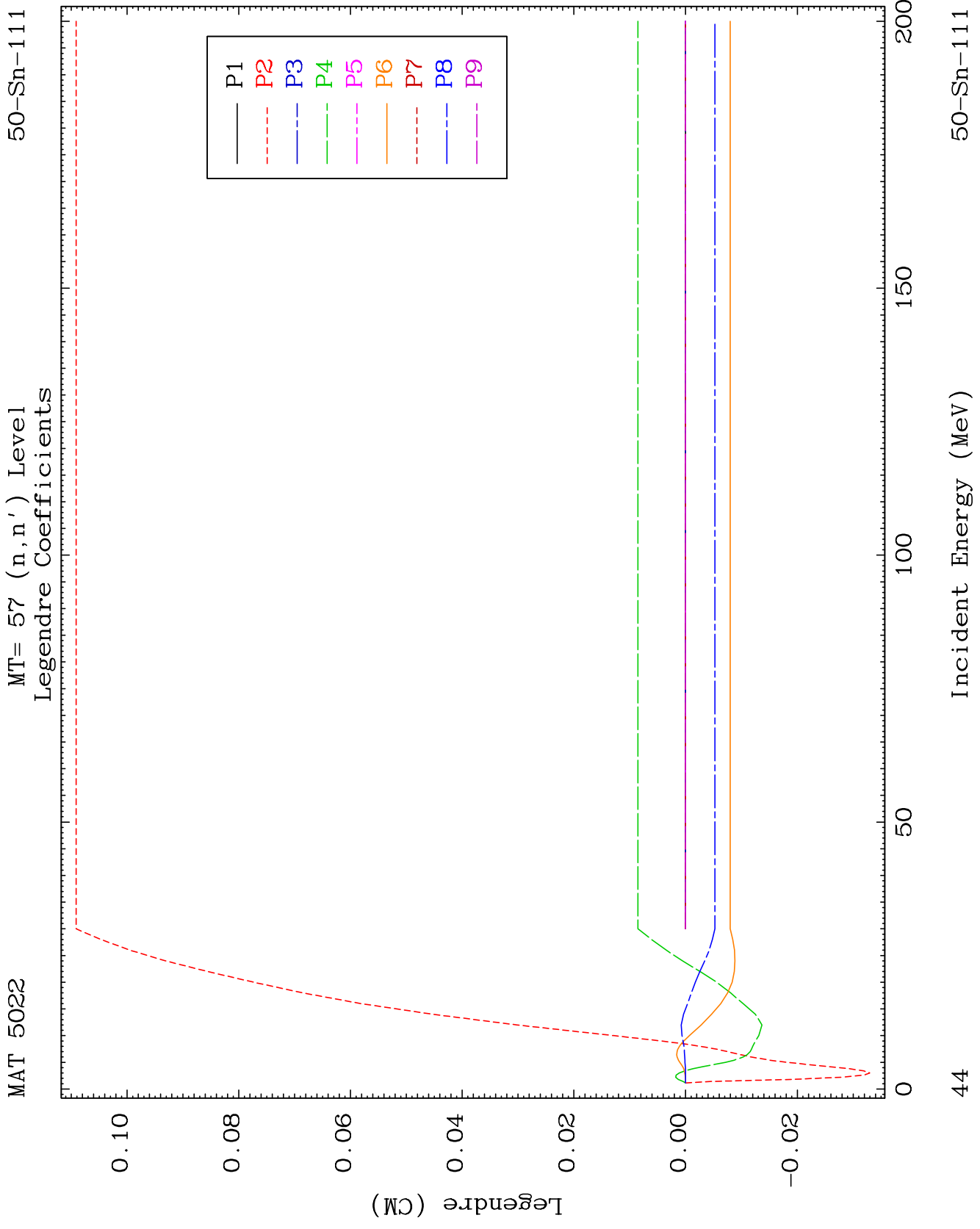




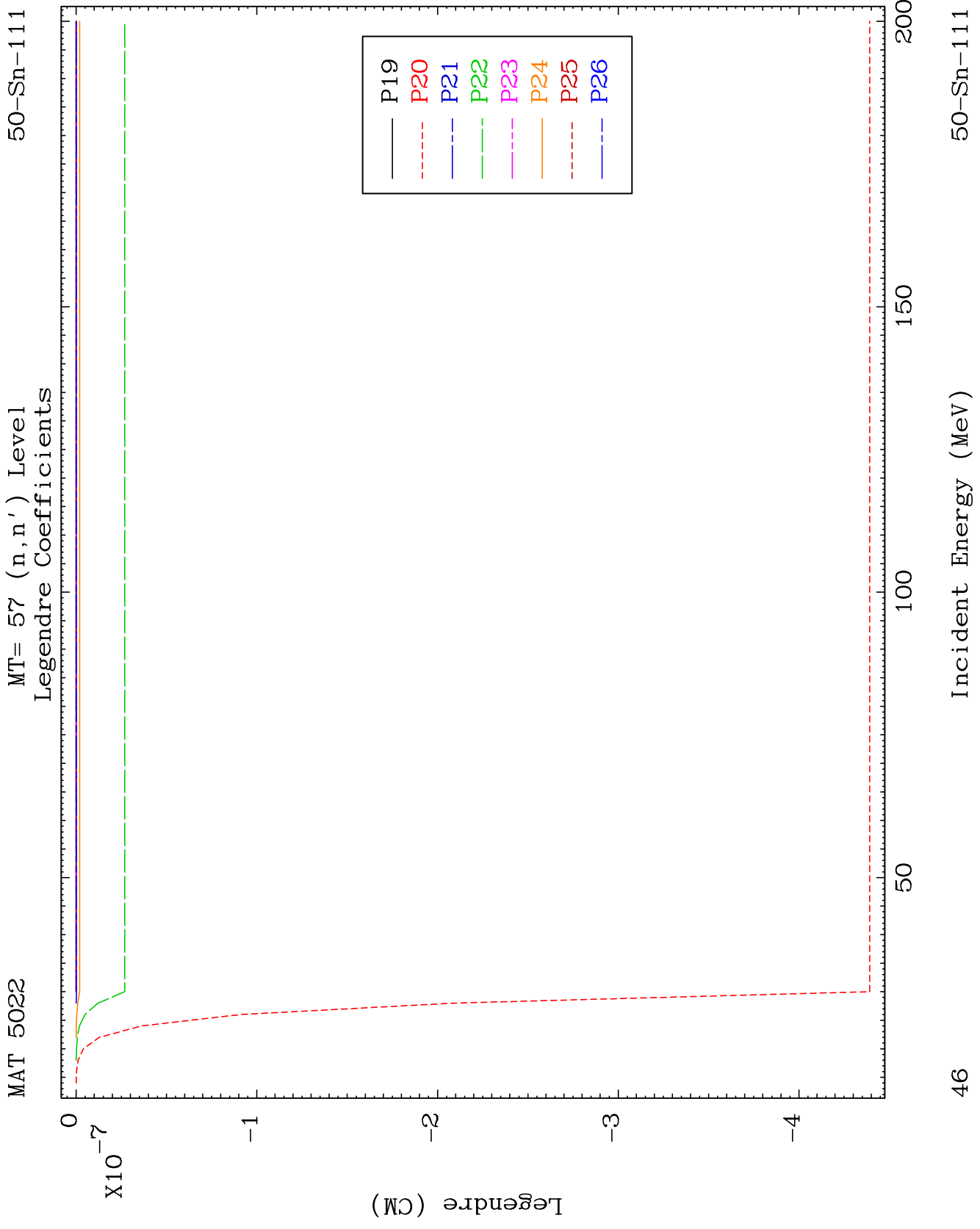


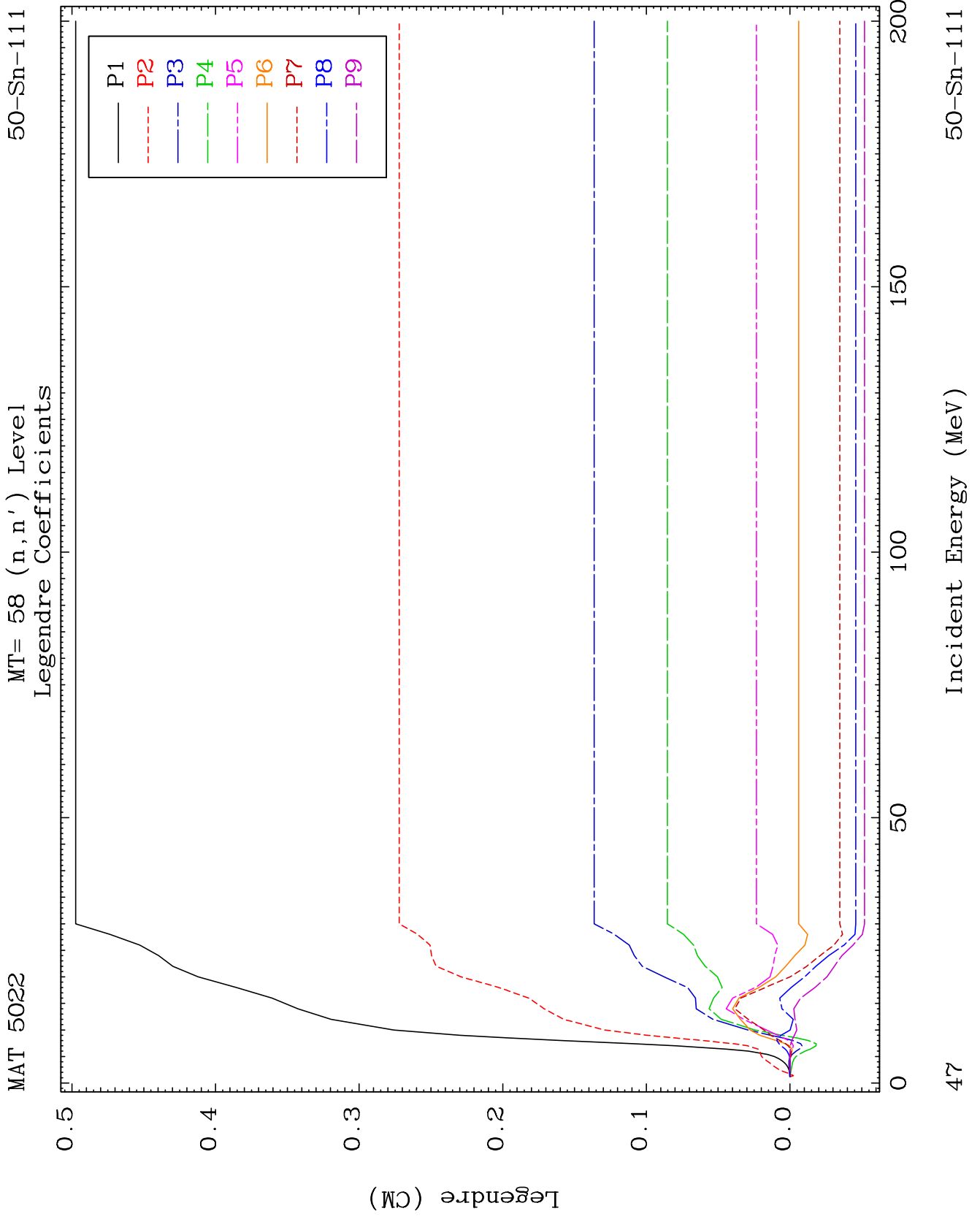




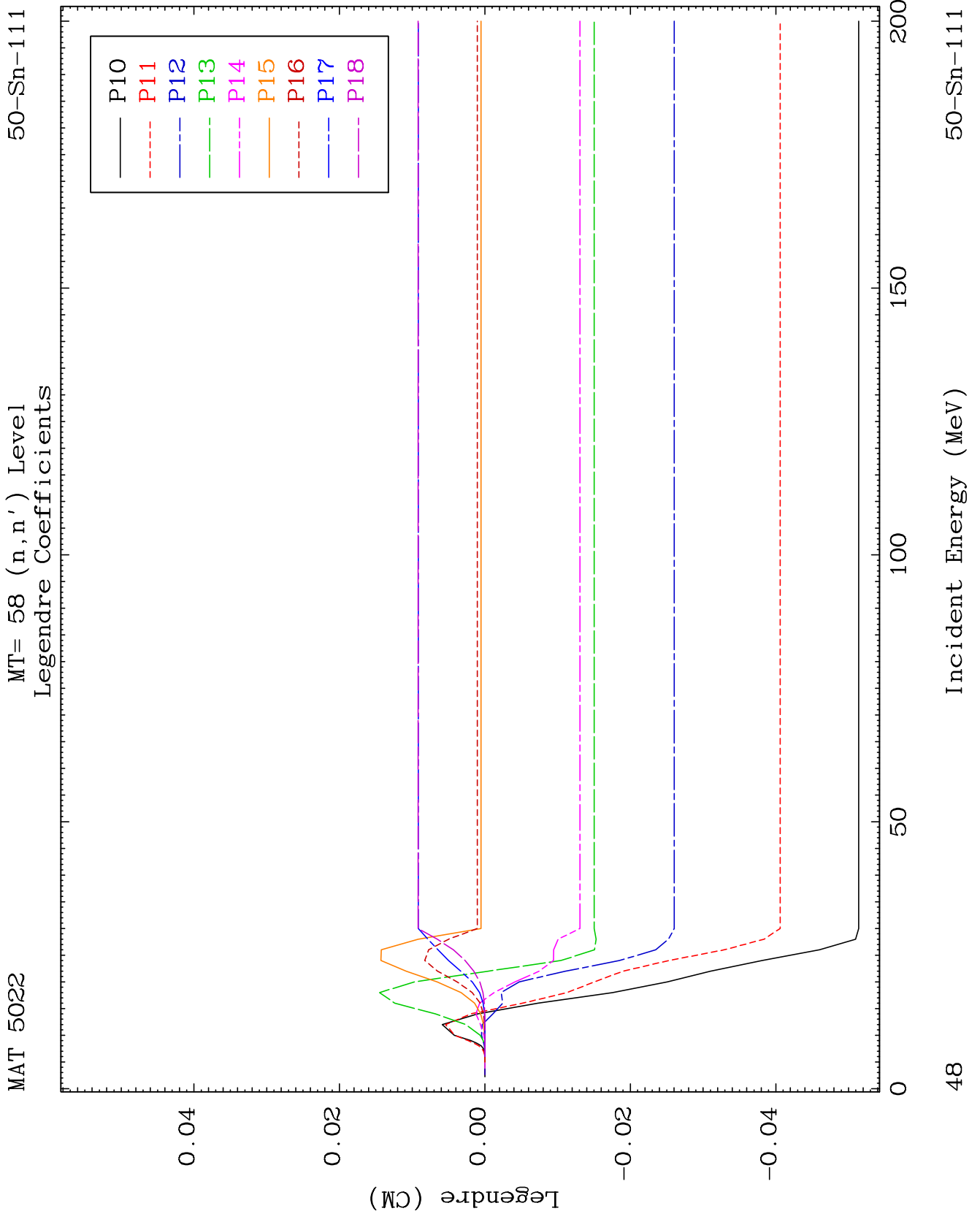


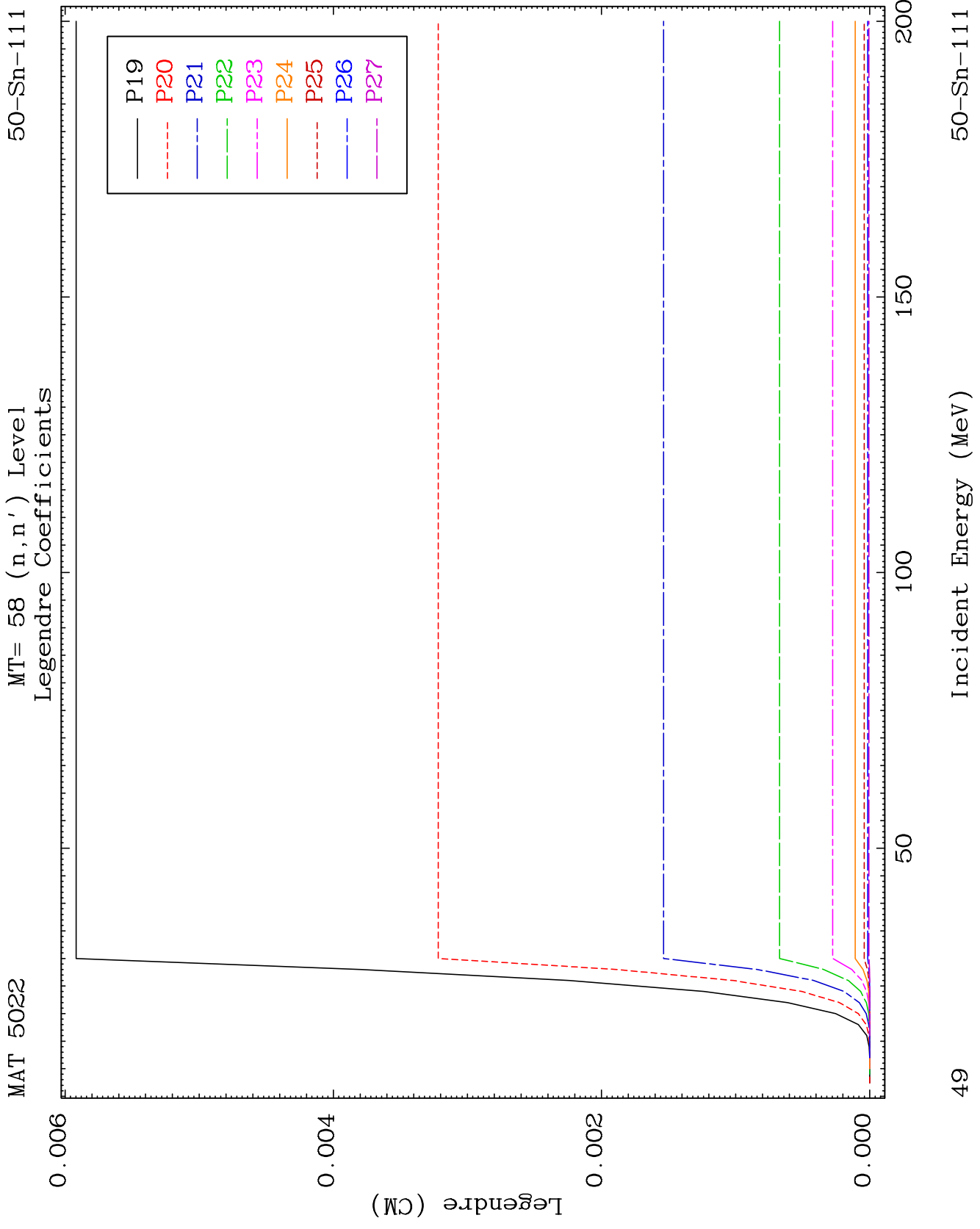


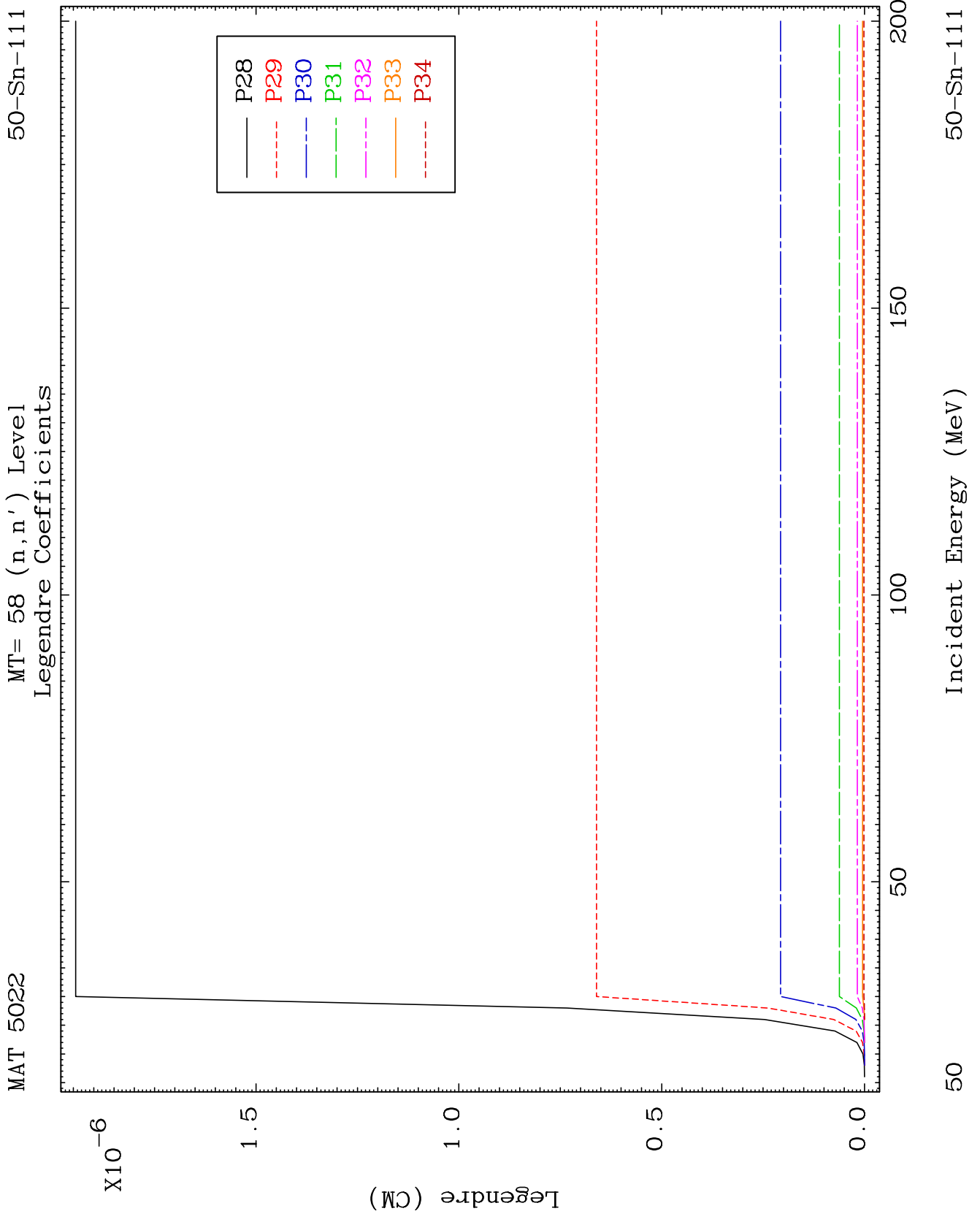


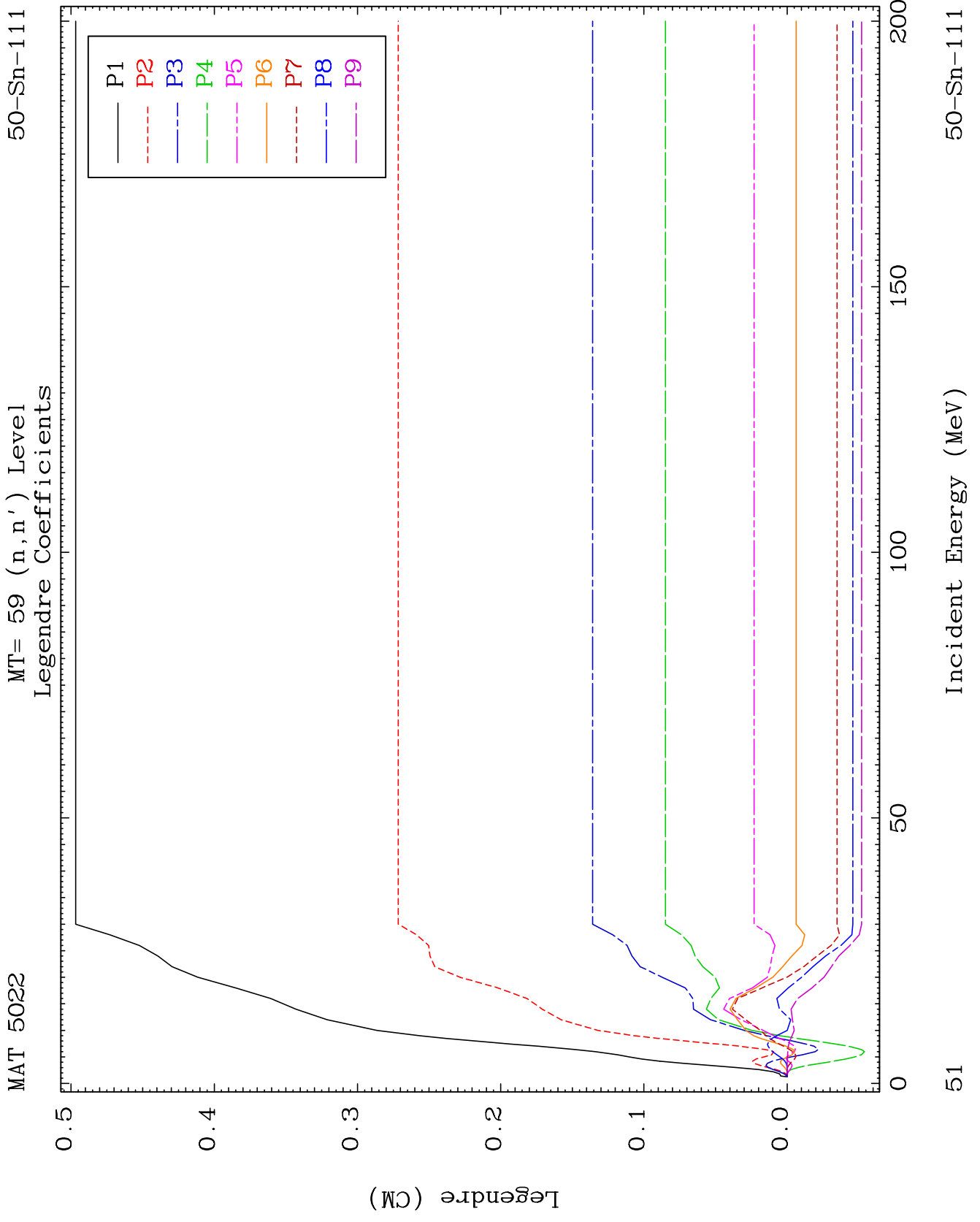


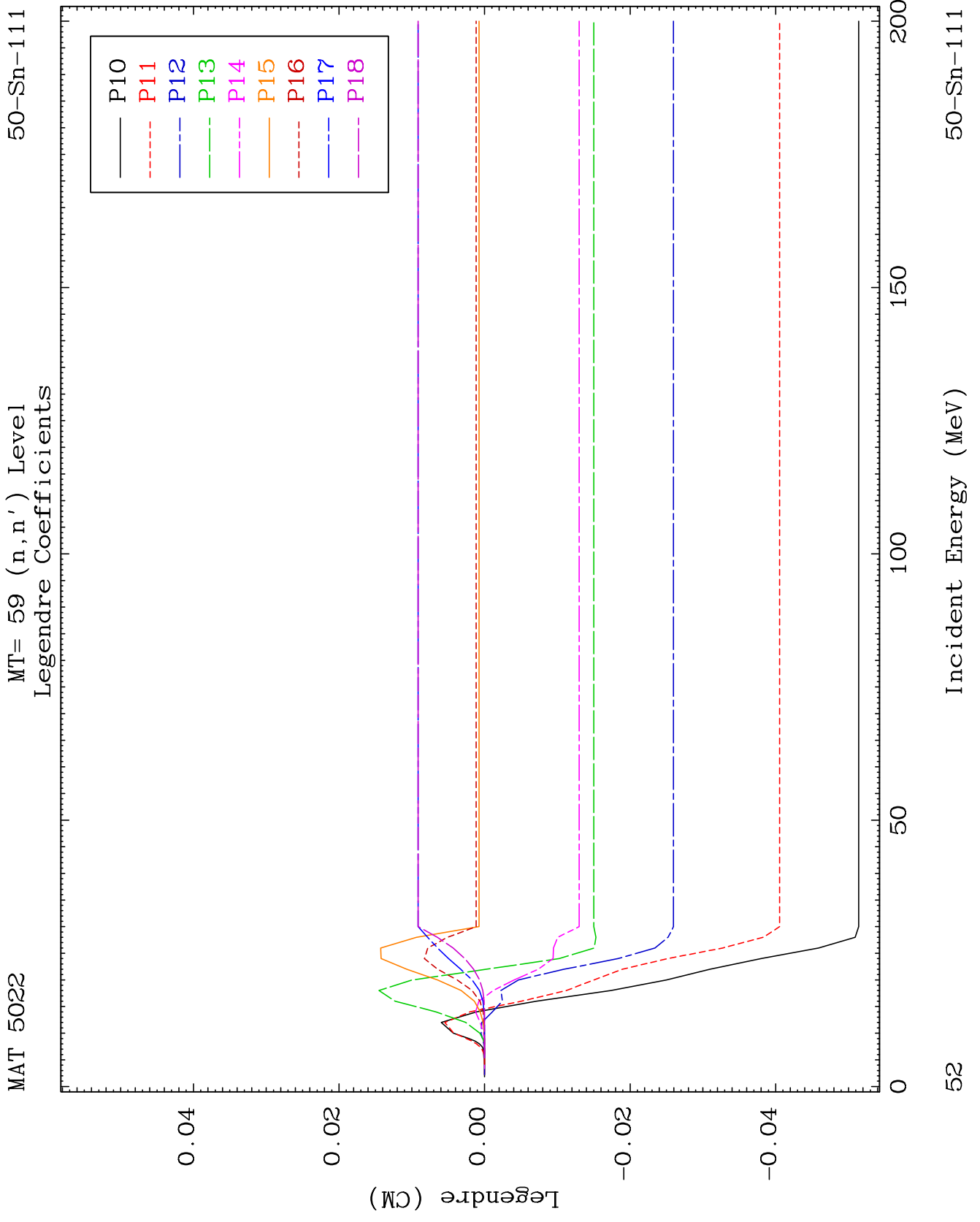


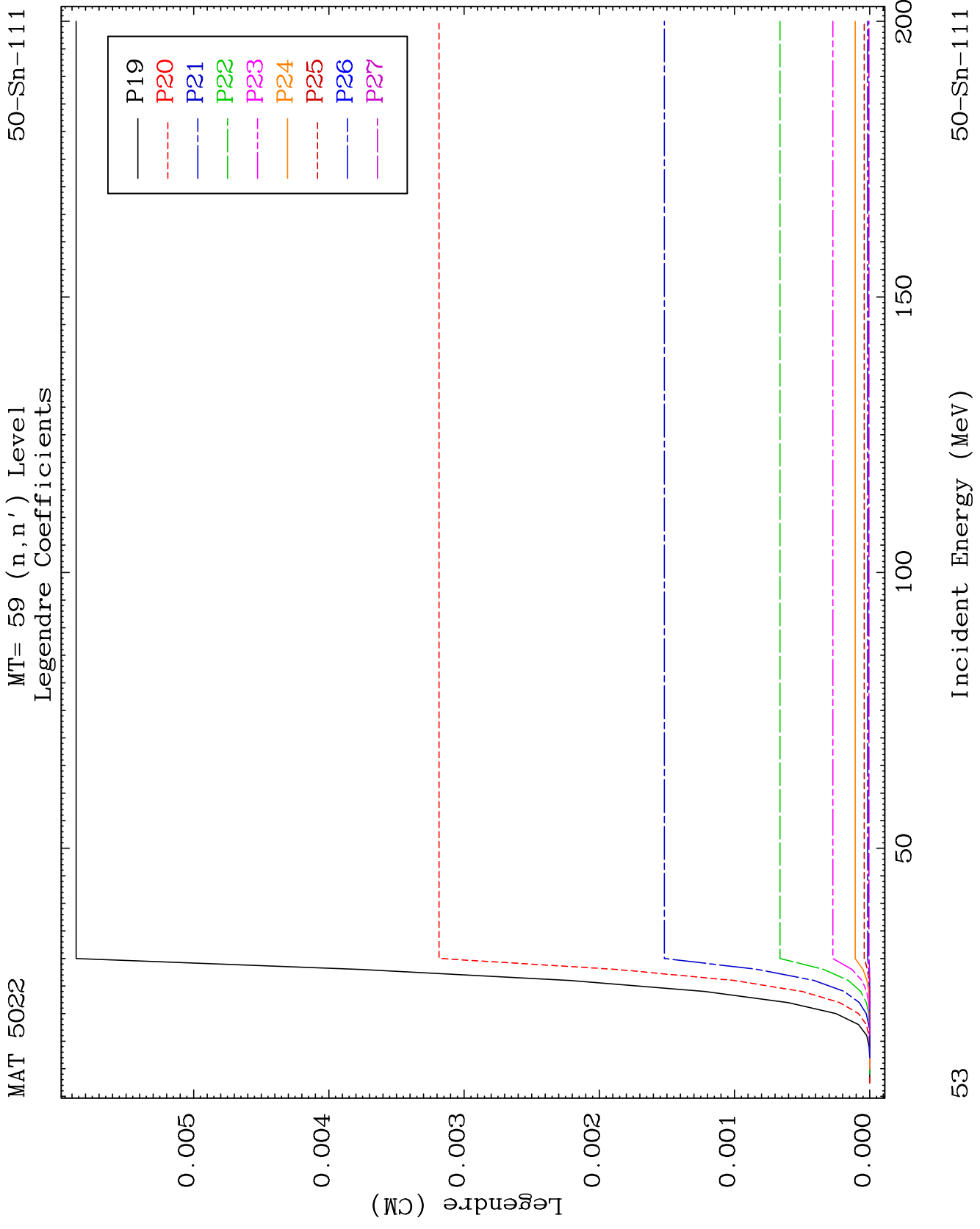








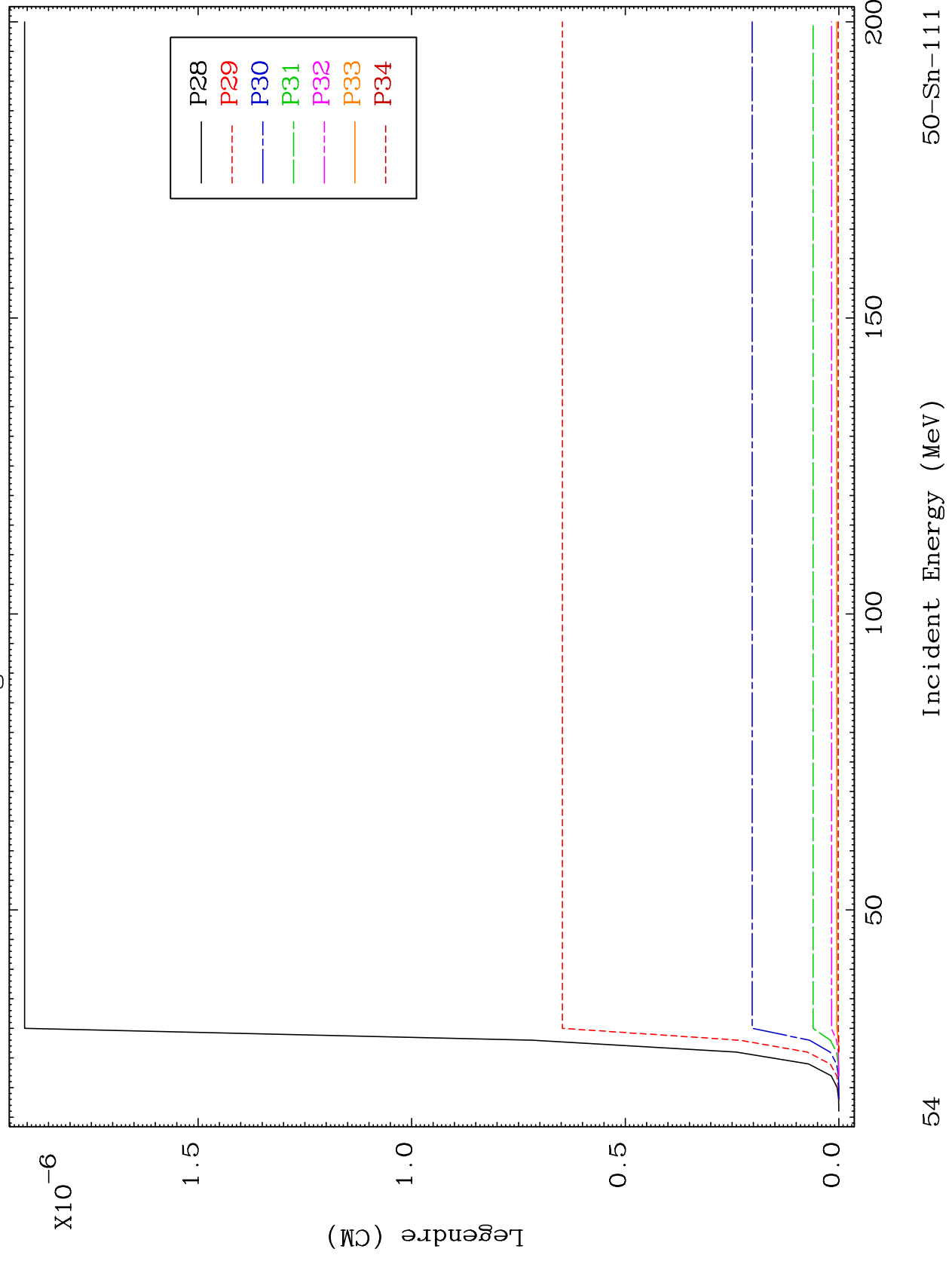




MAT 5022

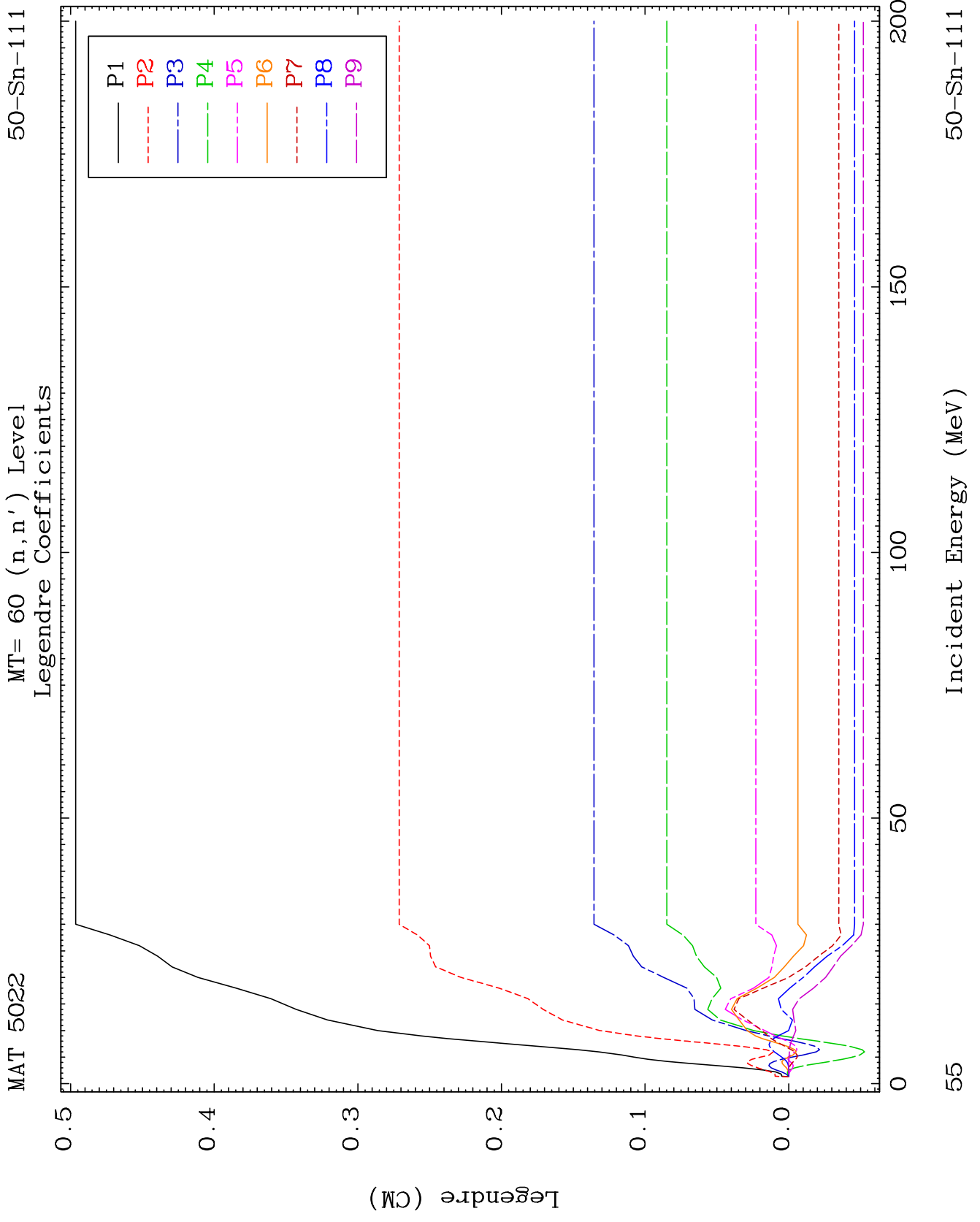
MT= 59 (n,n') Level  
Legendre Coefficients

50-Sn-111

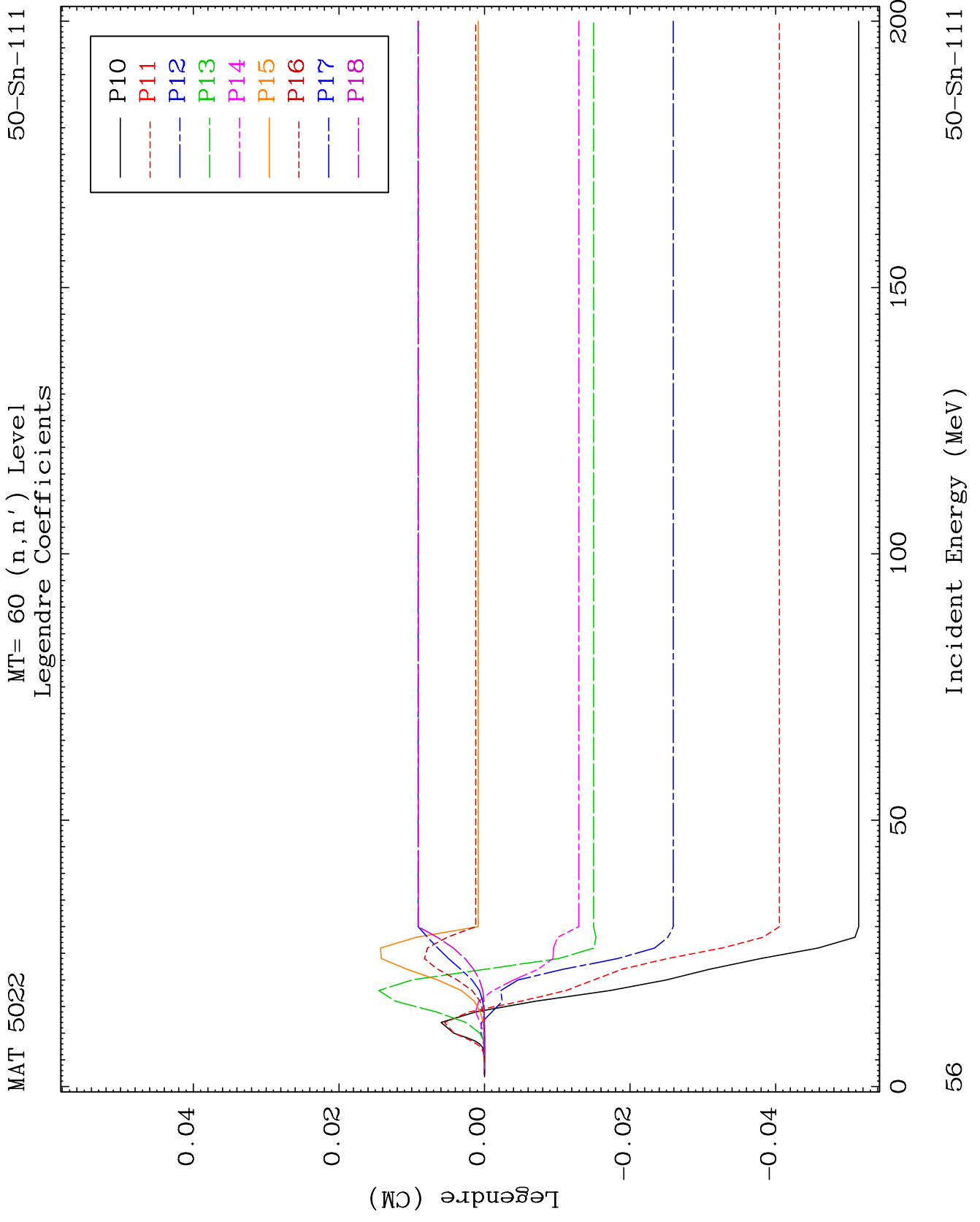


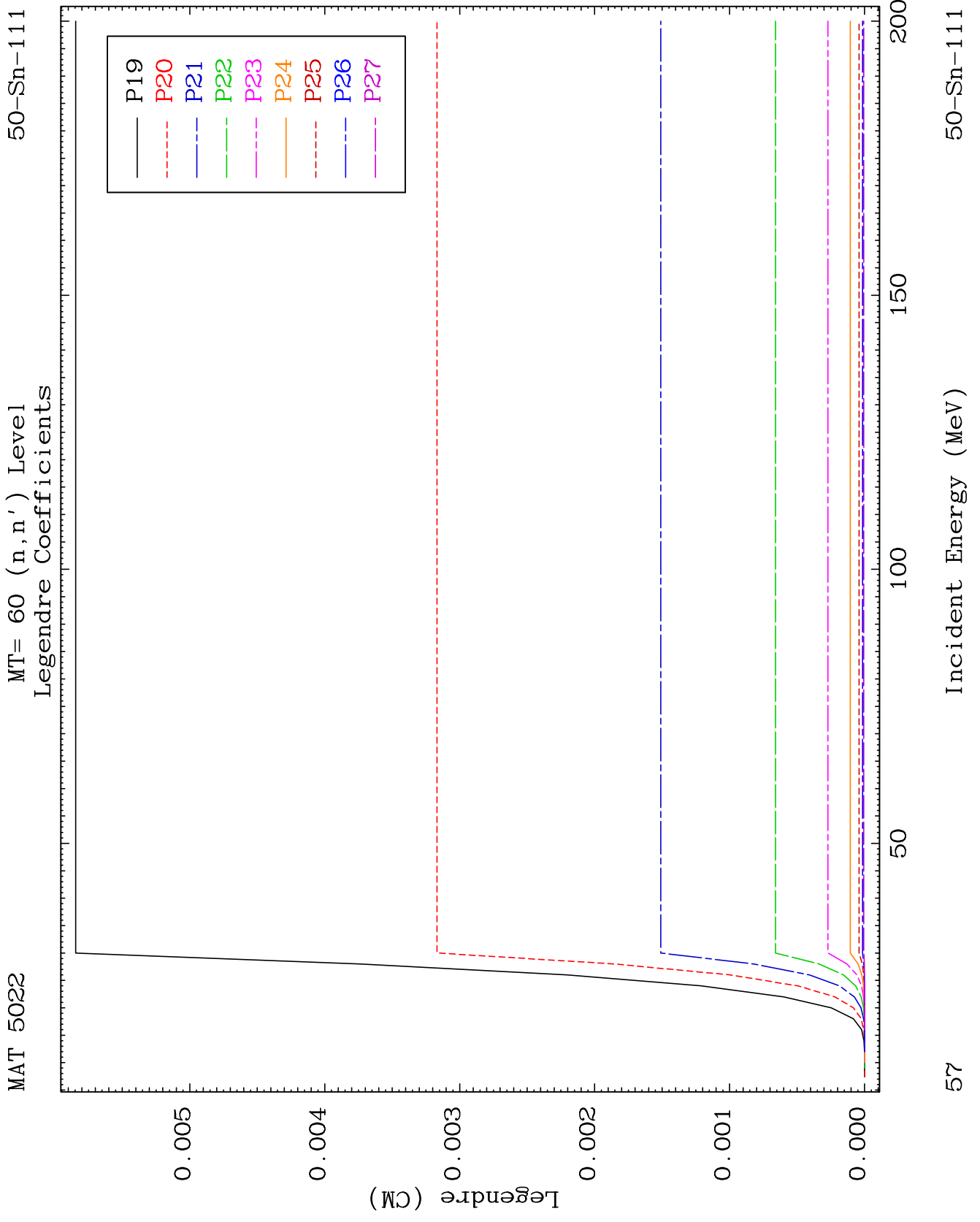
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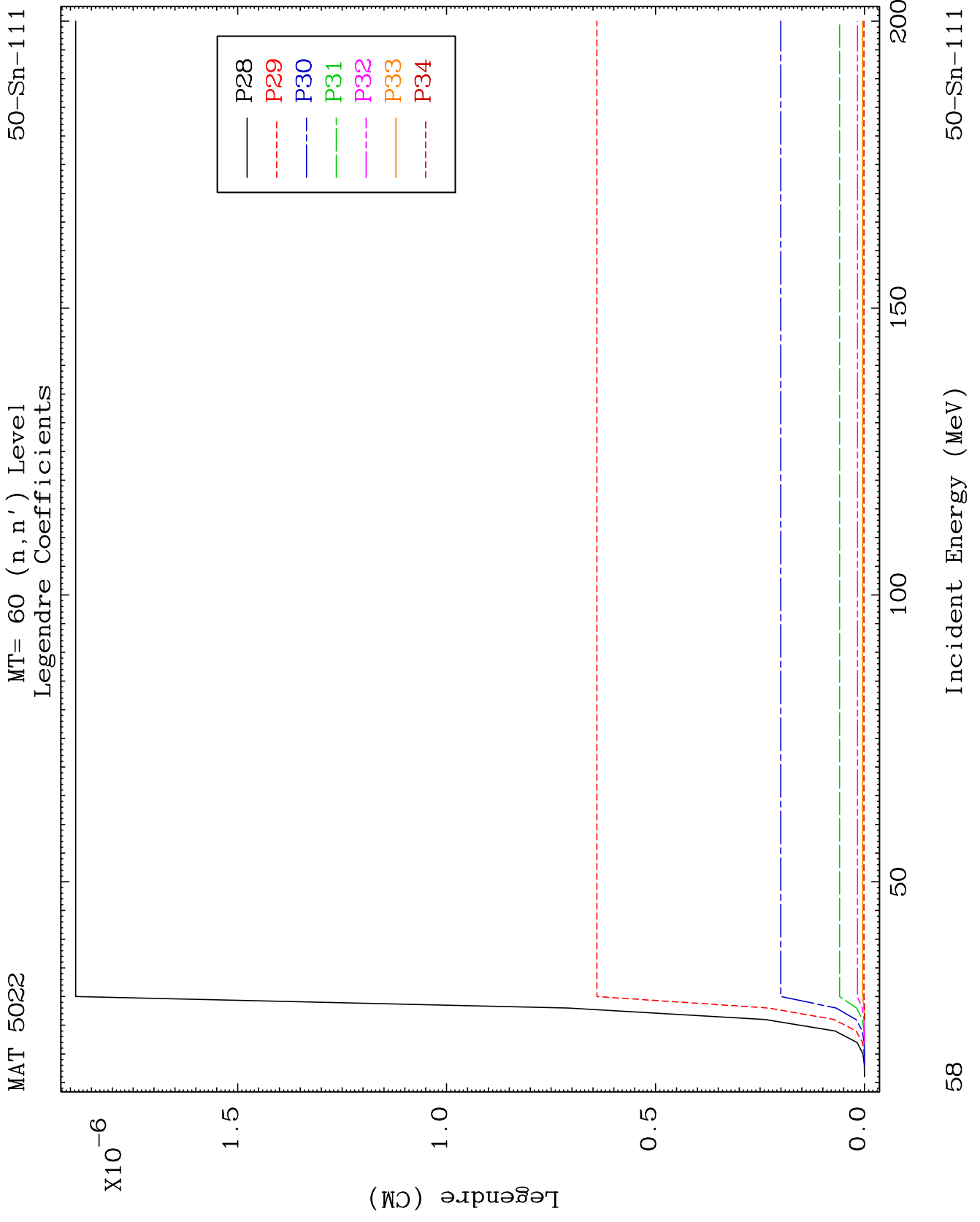
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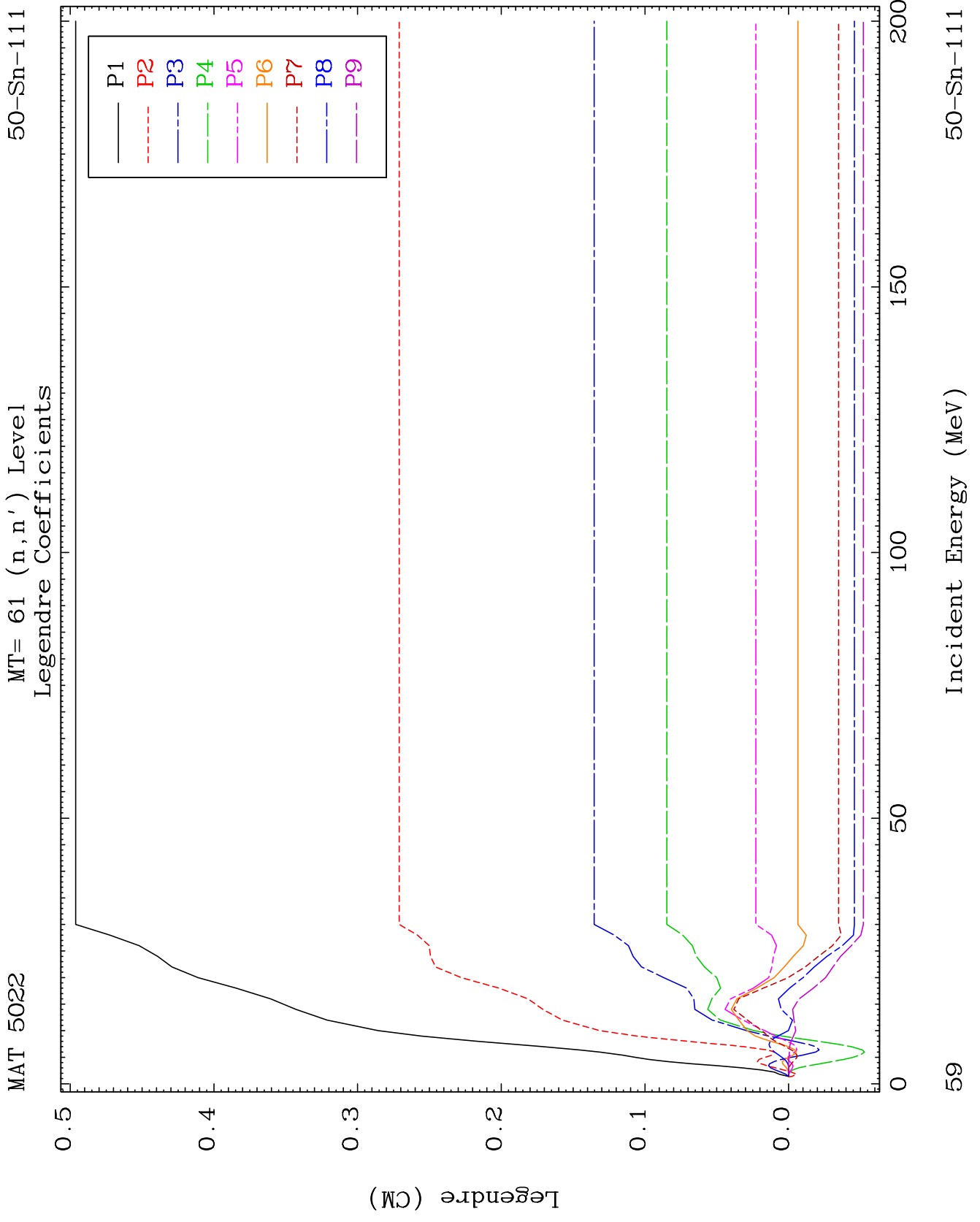




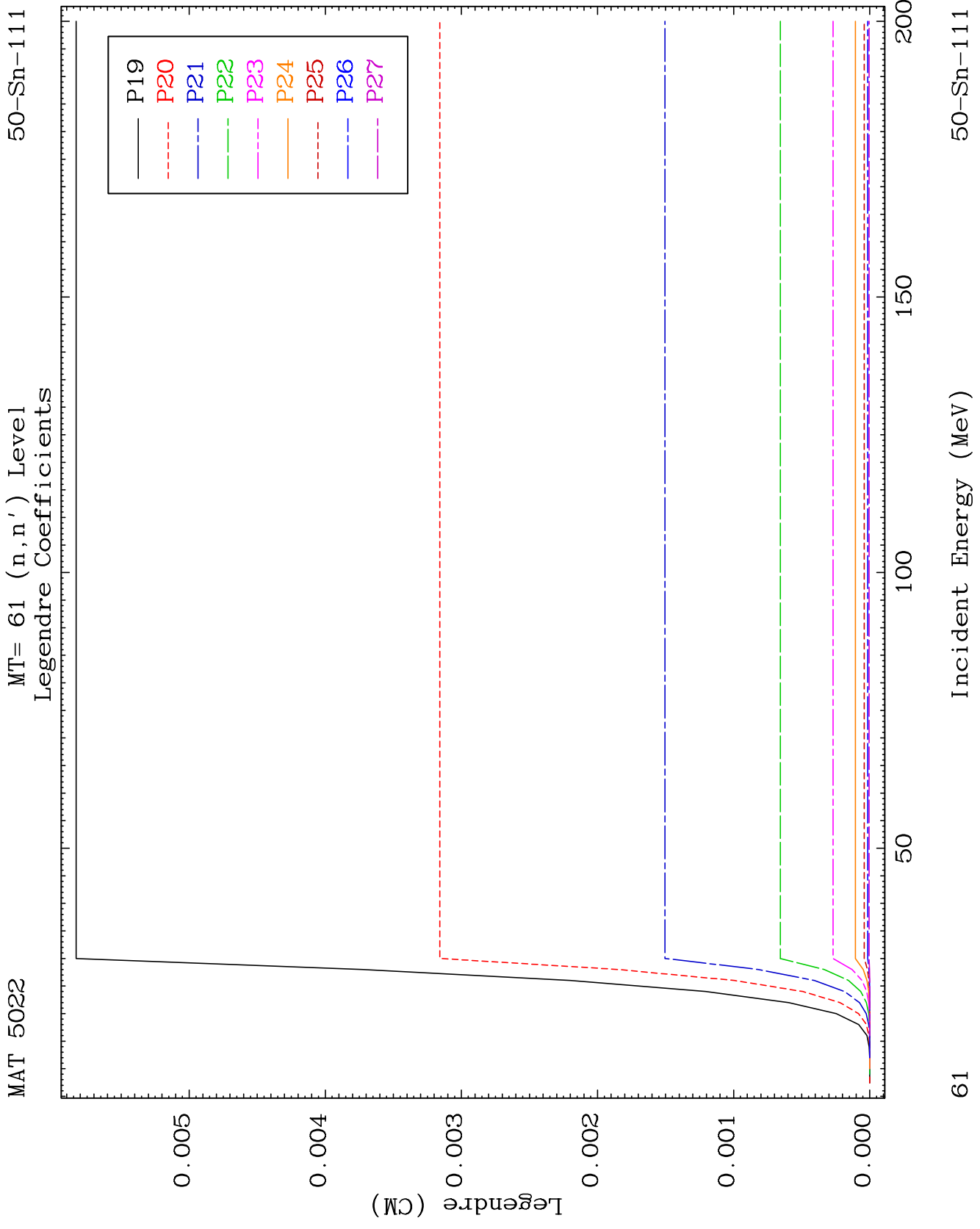








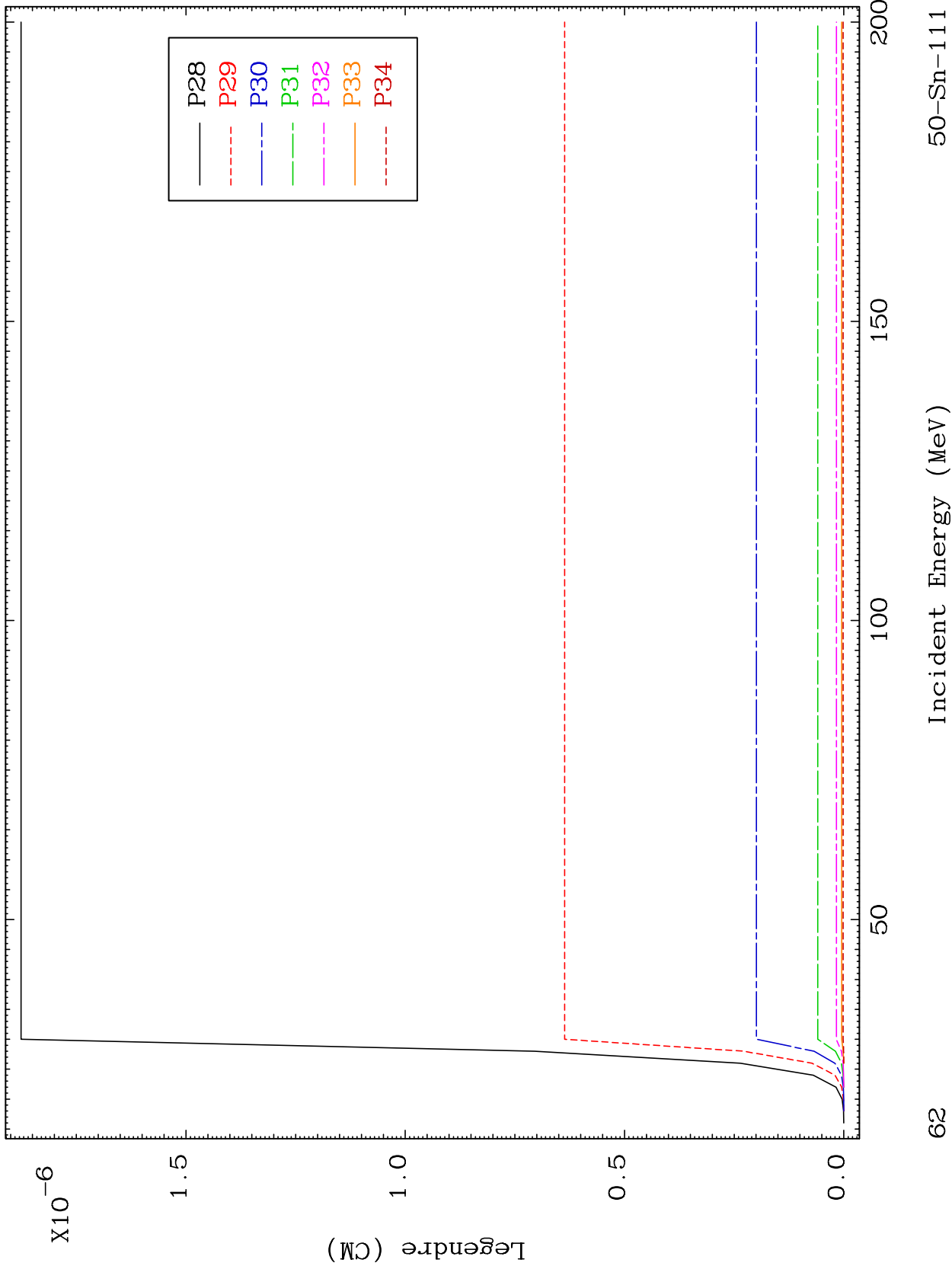


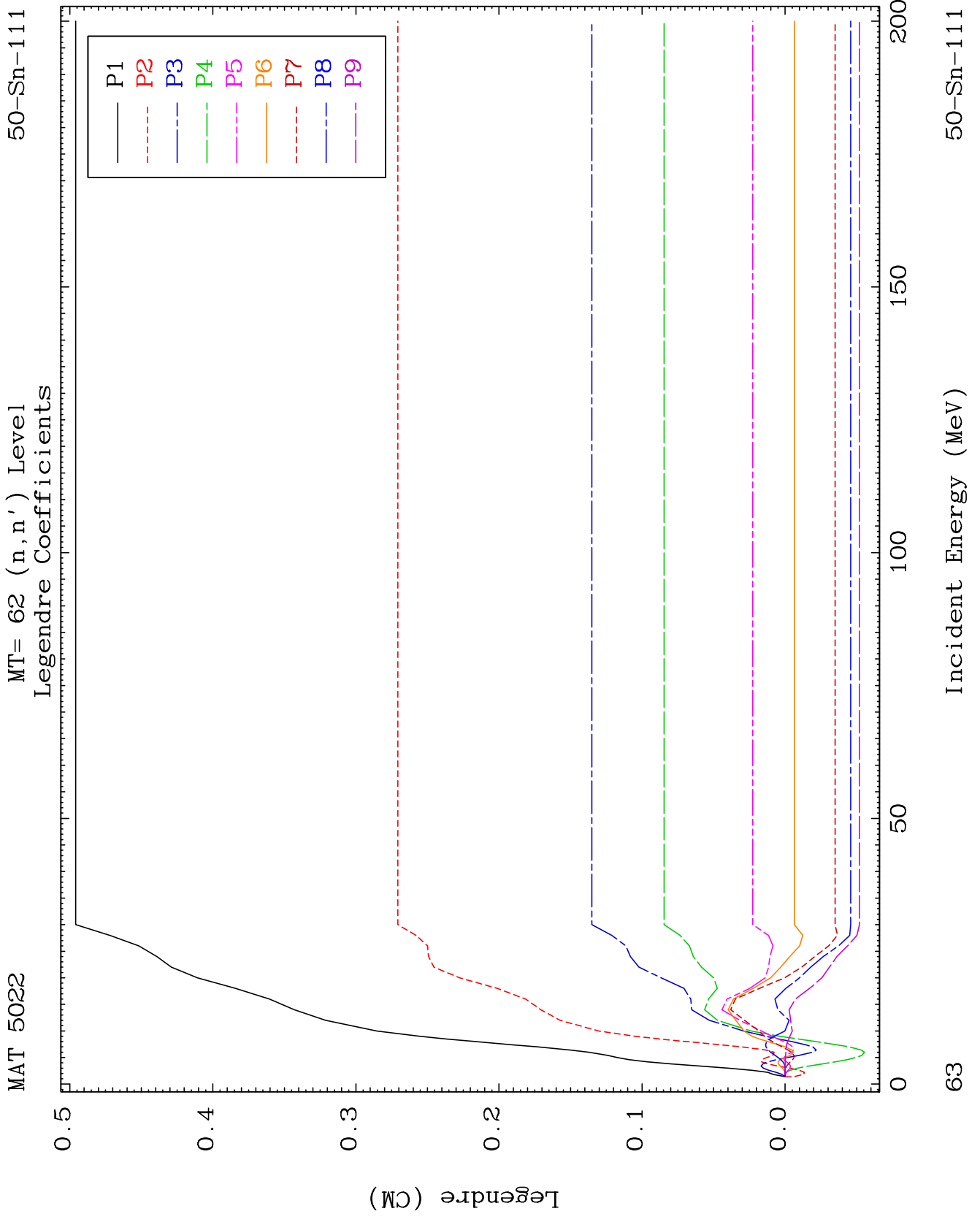


MAT 5022

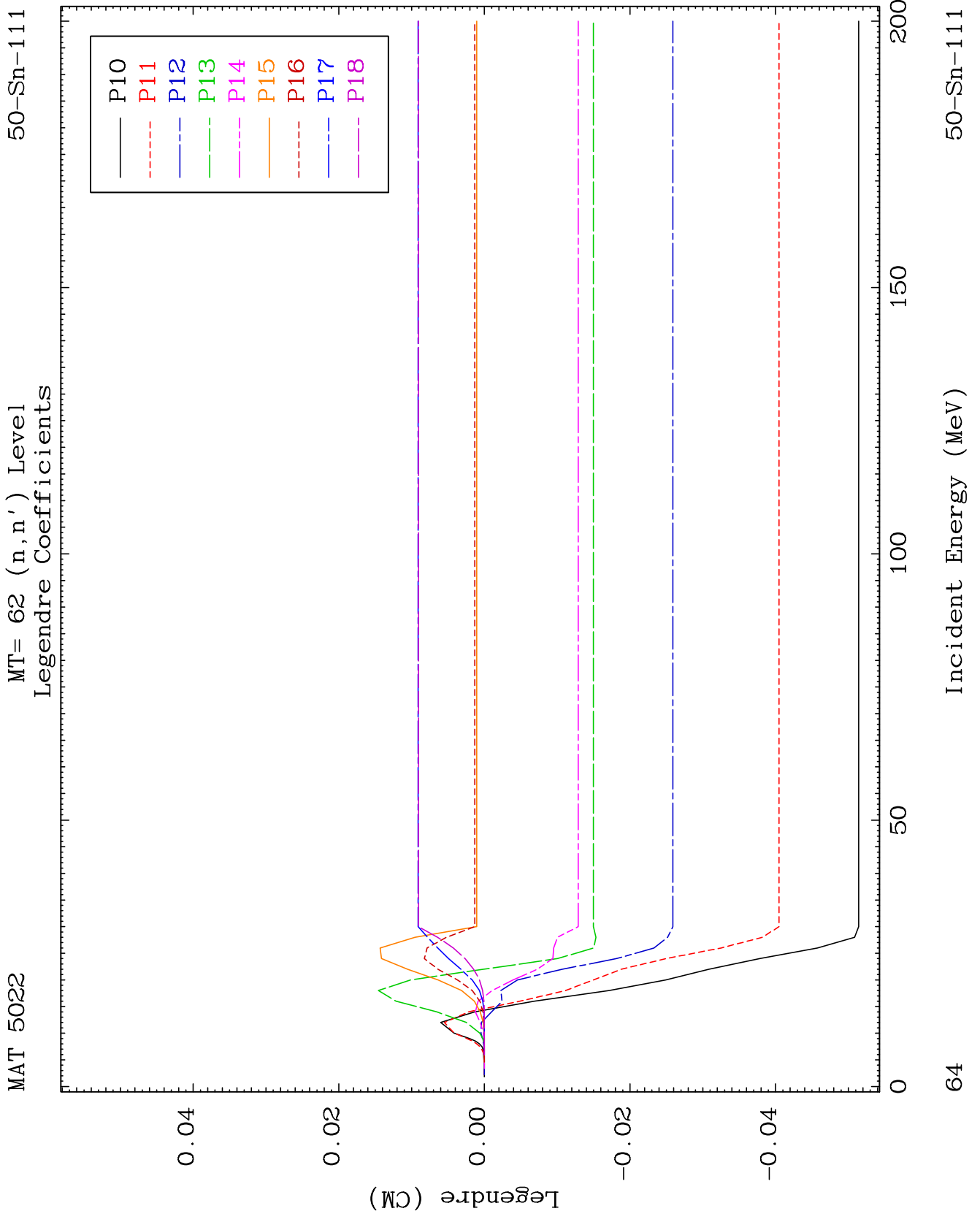
MT= 61 (n,n') Level  
Legendre Coefficients

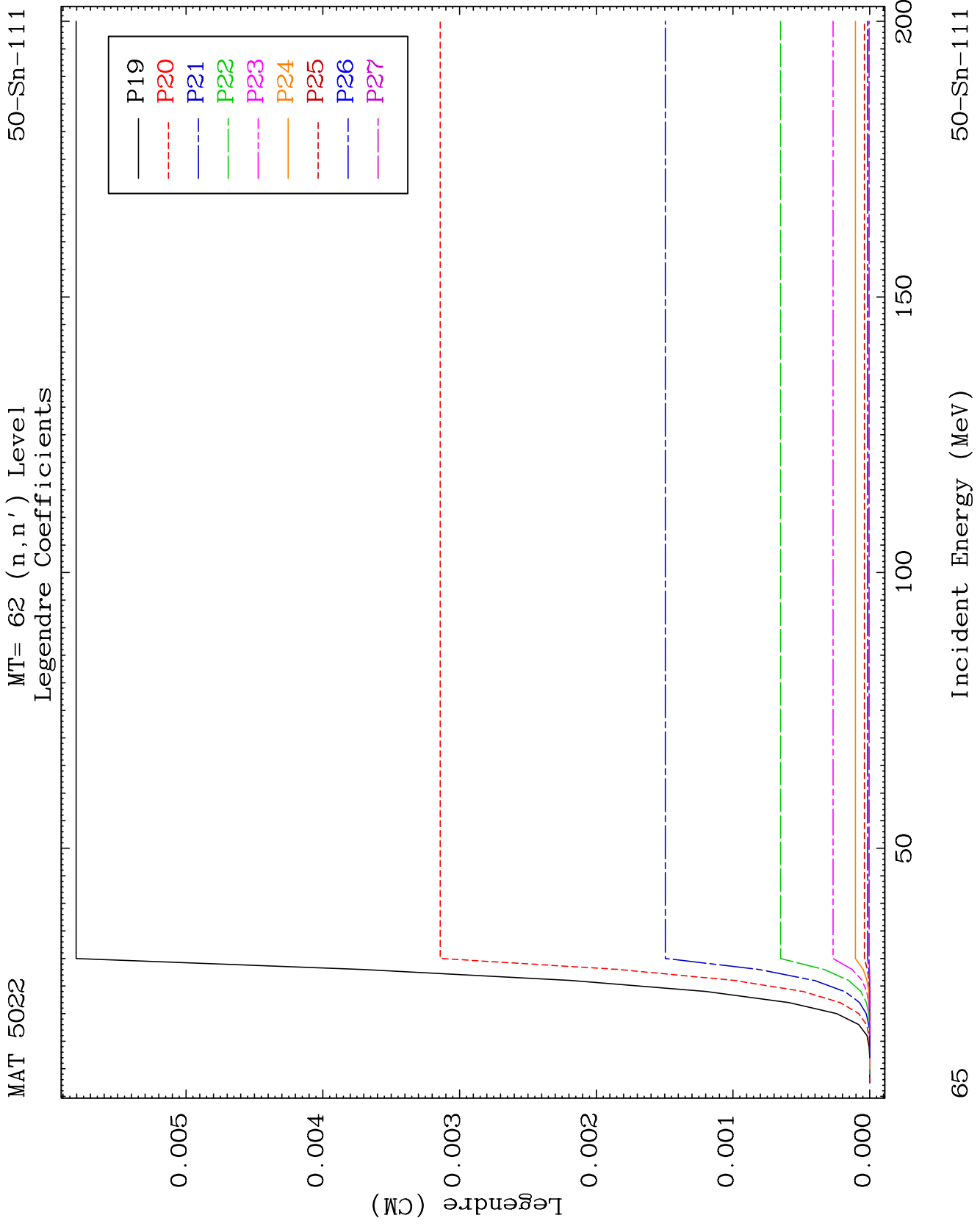
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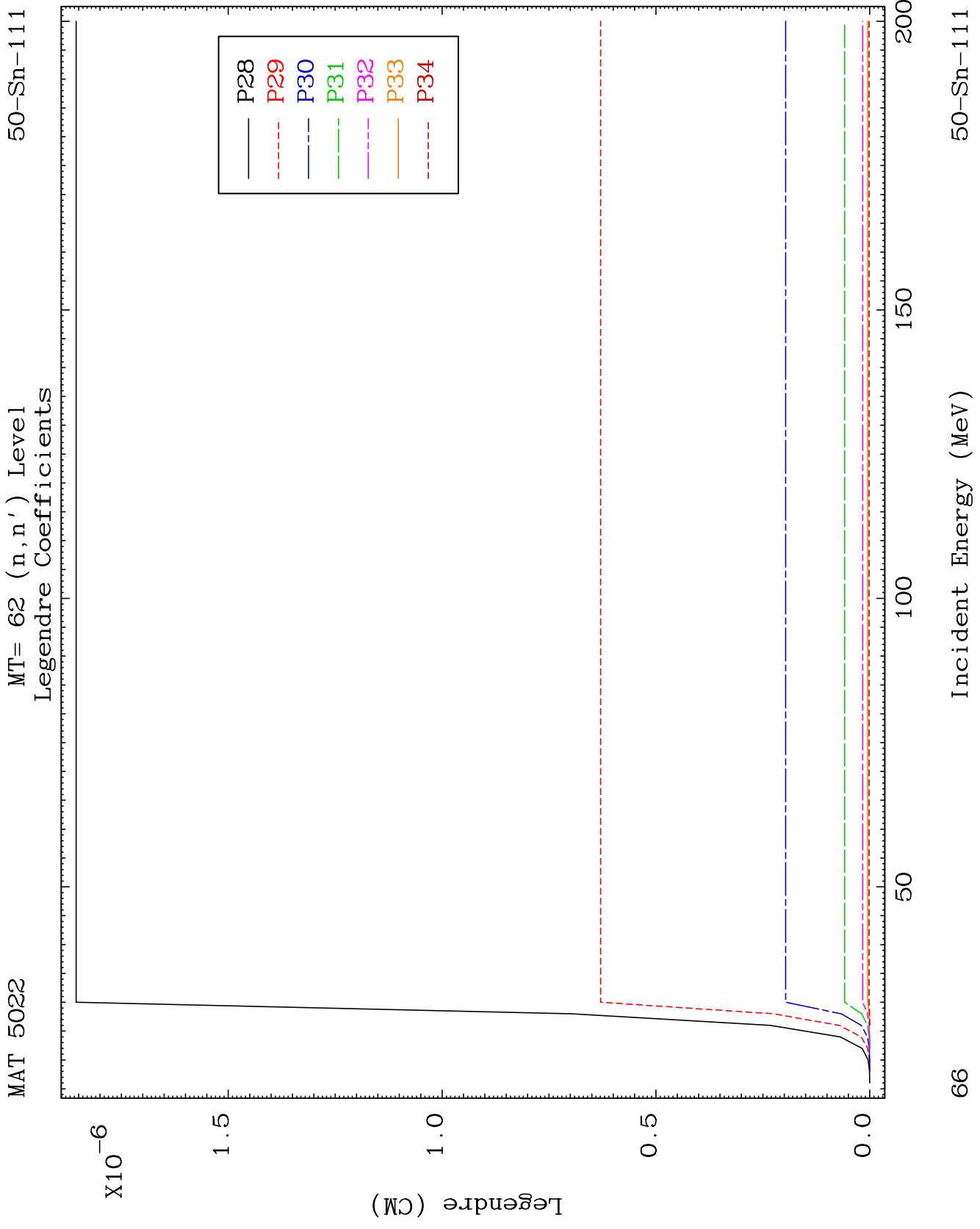




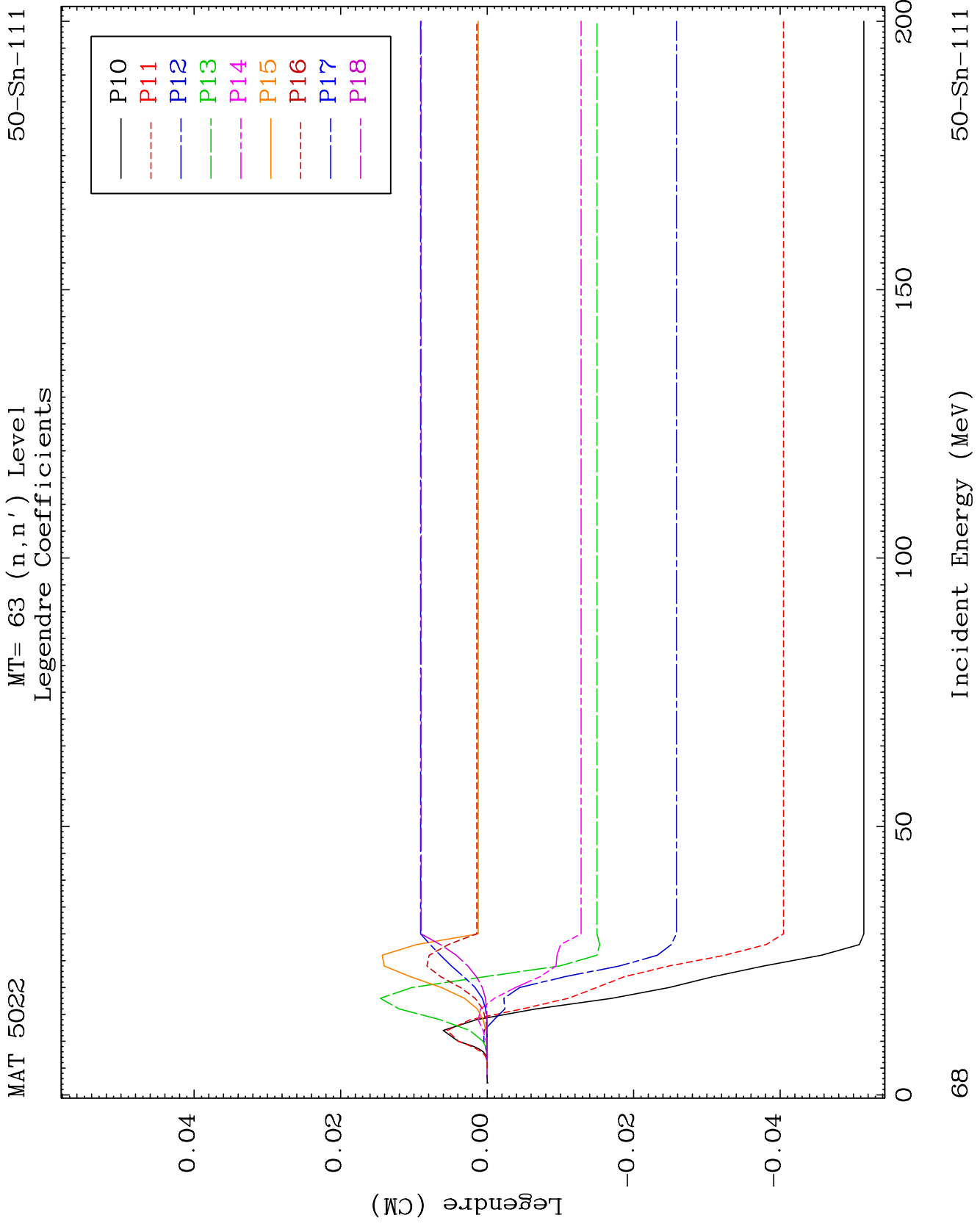




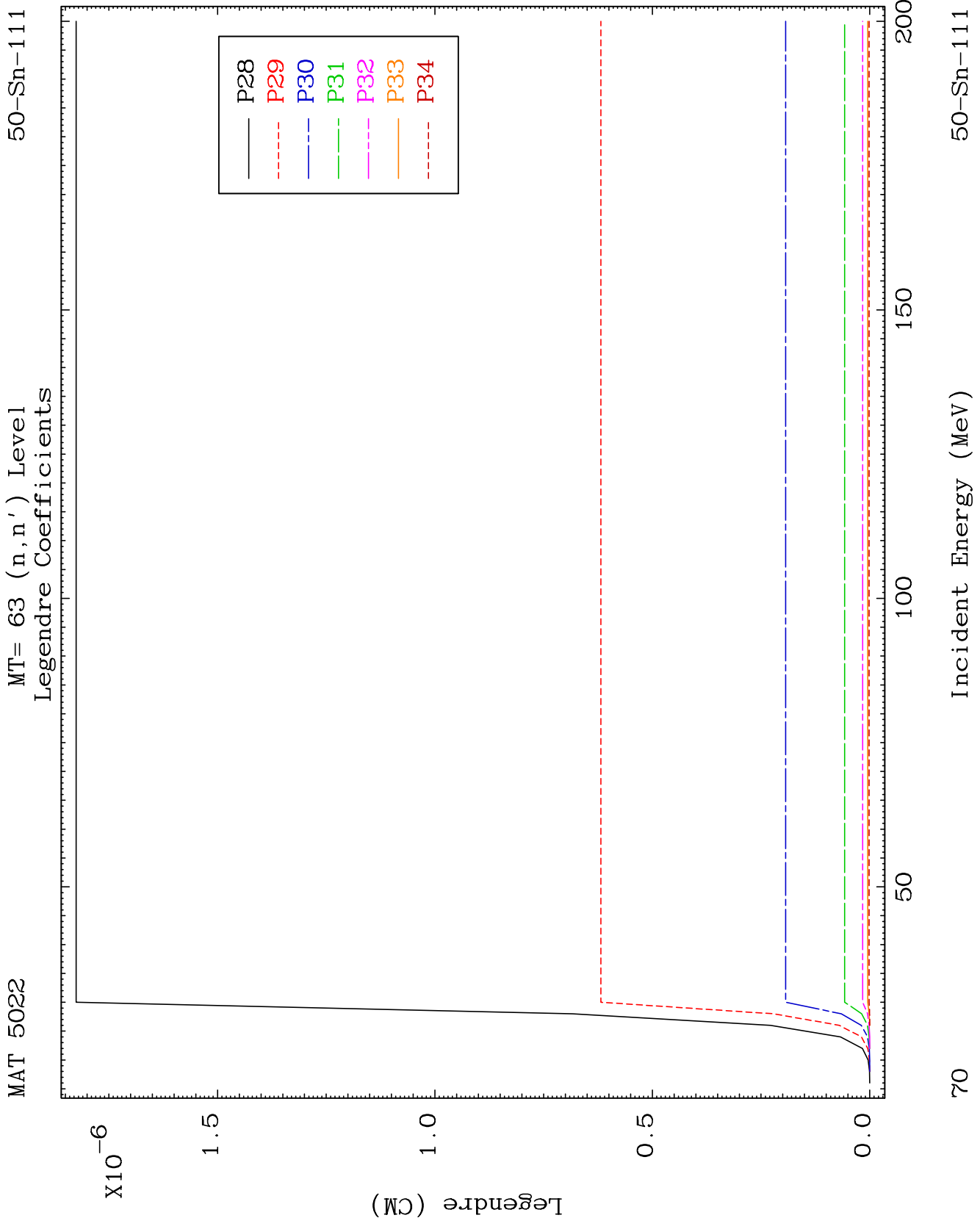


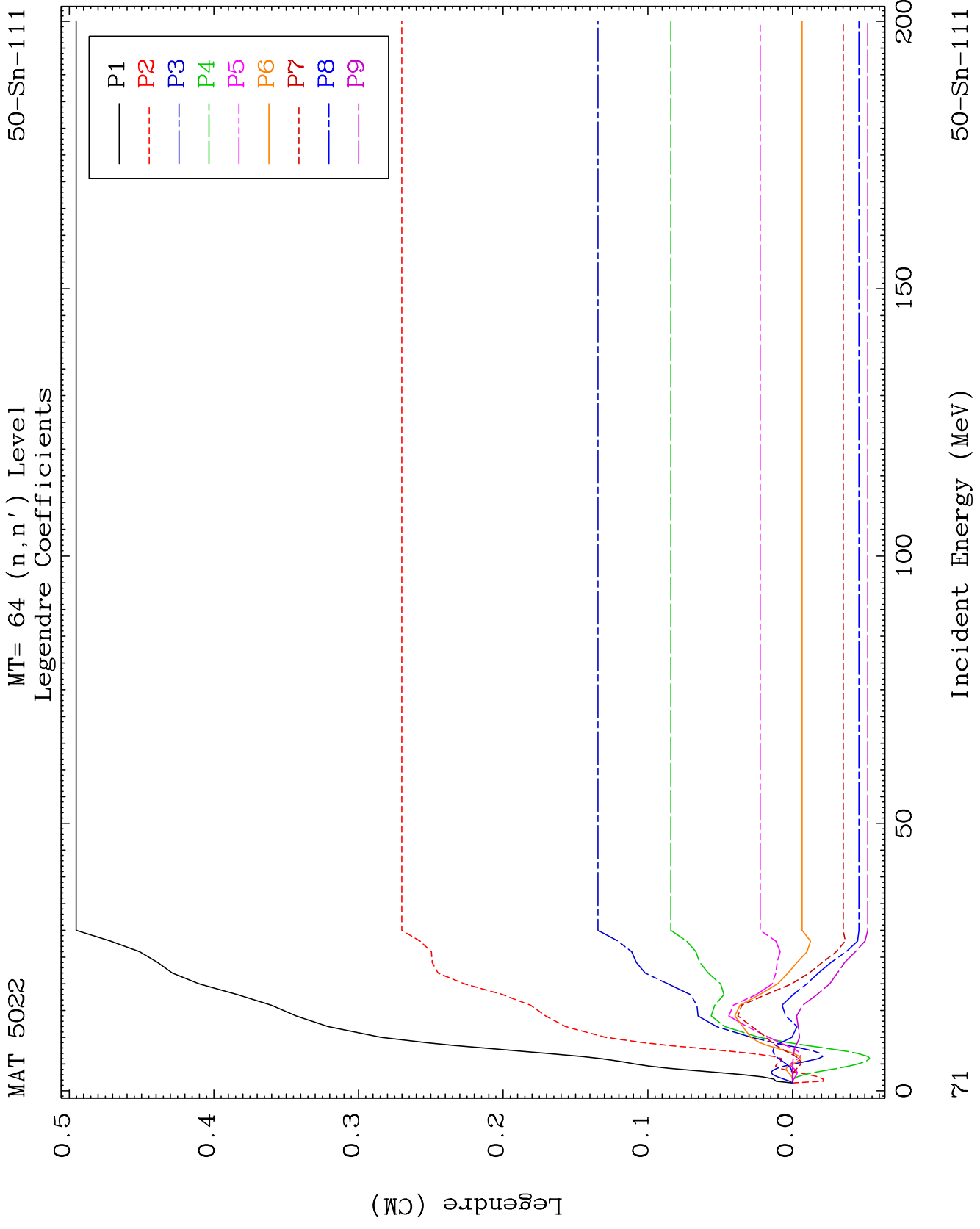




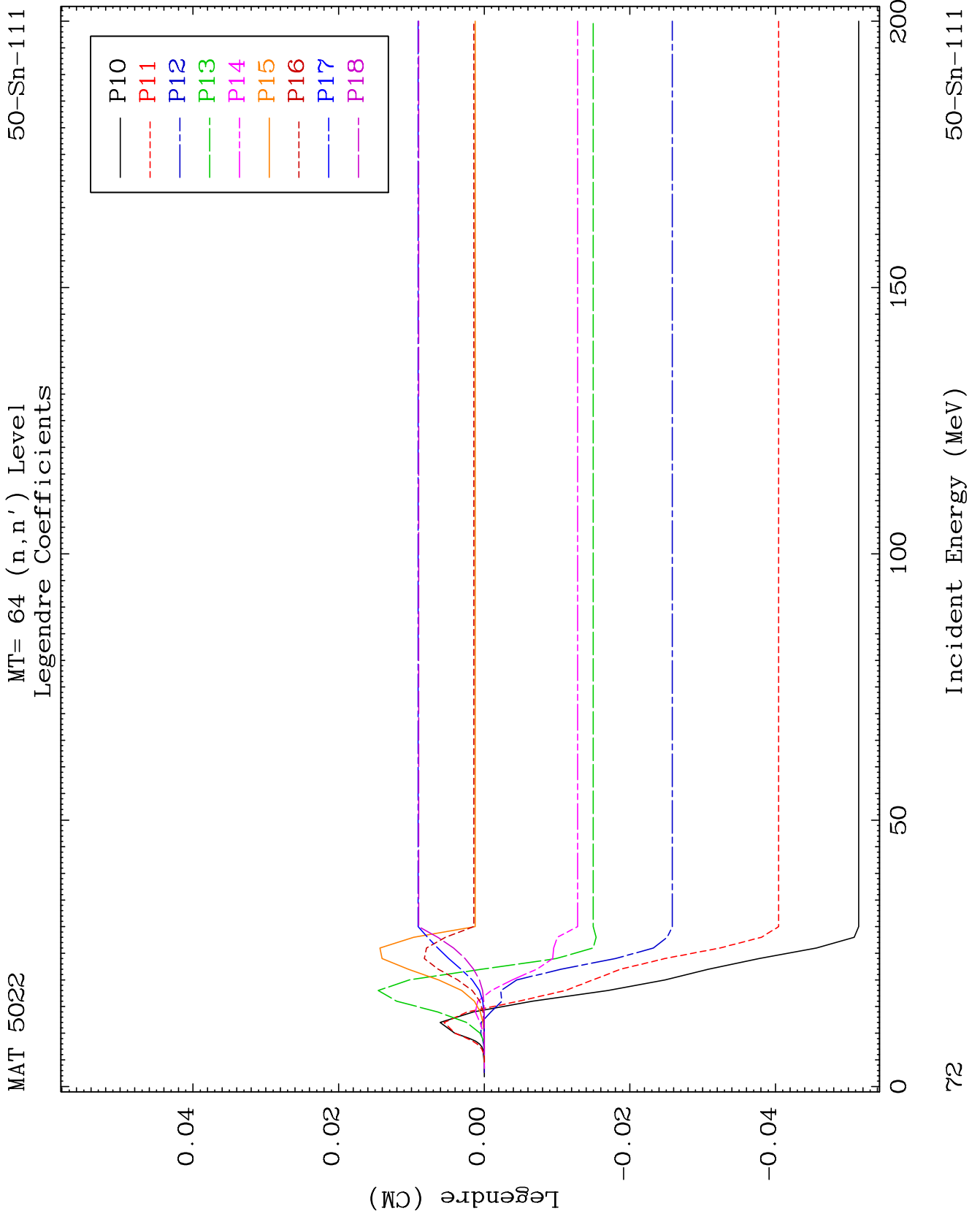


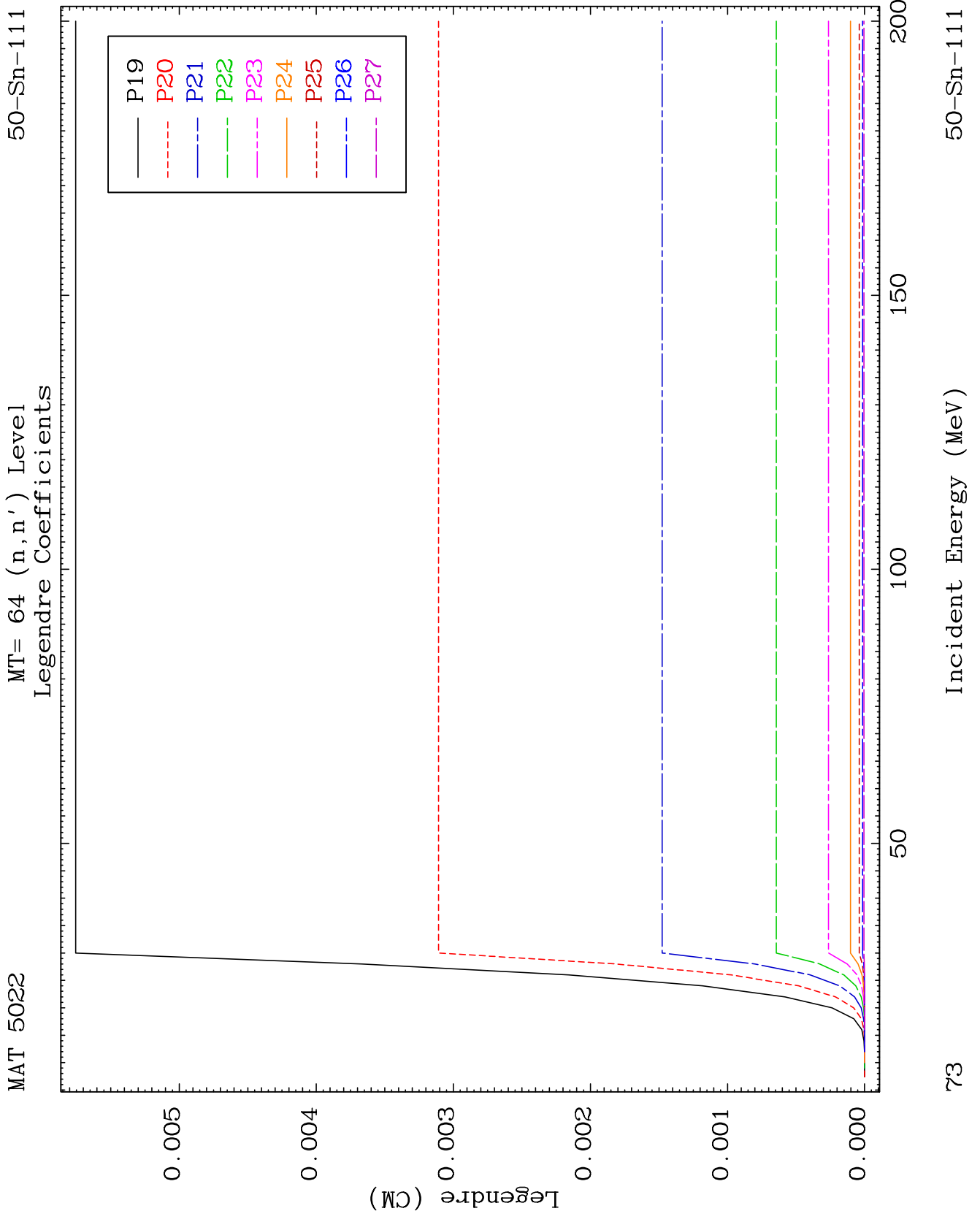








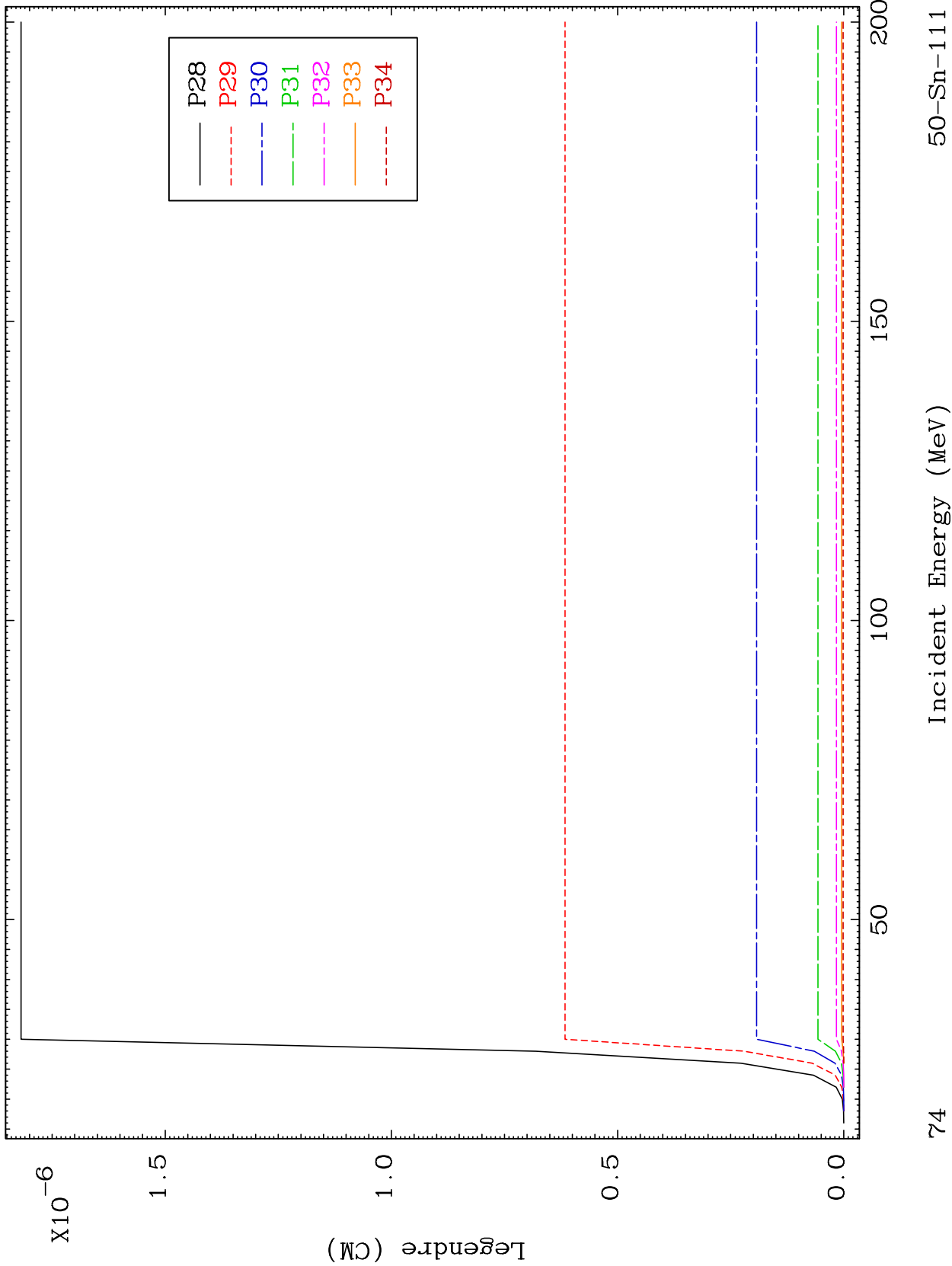




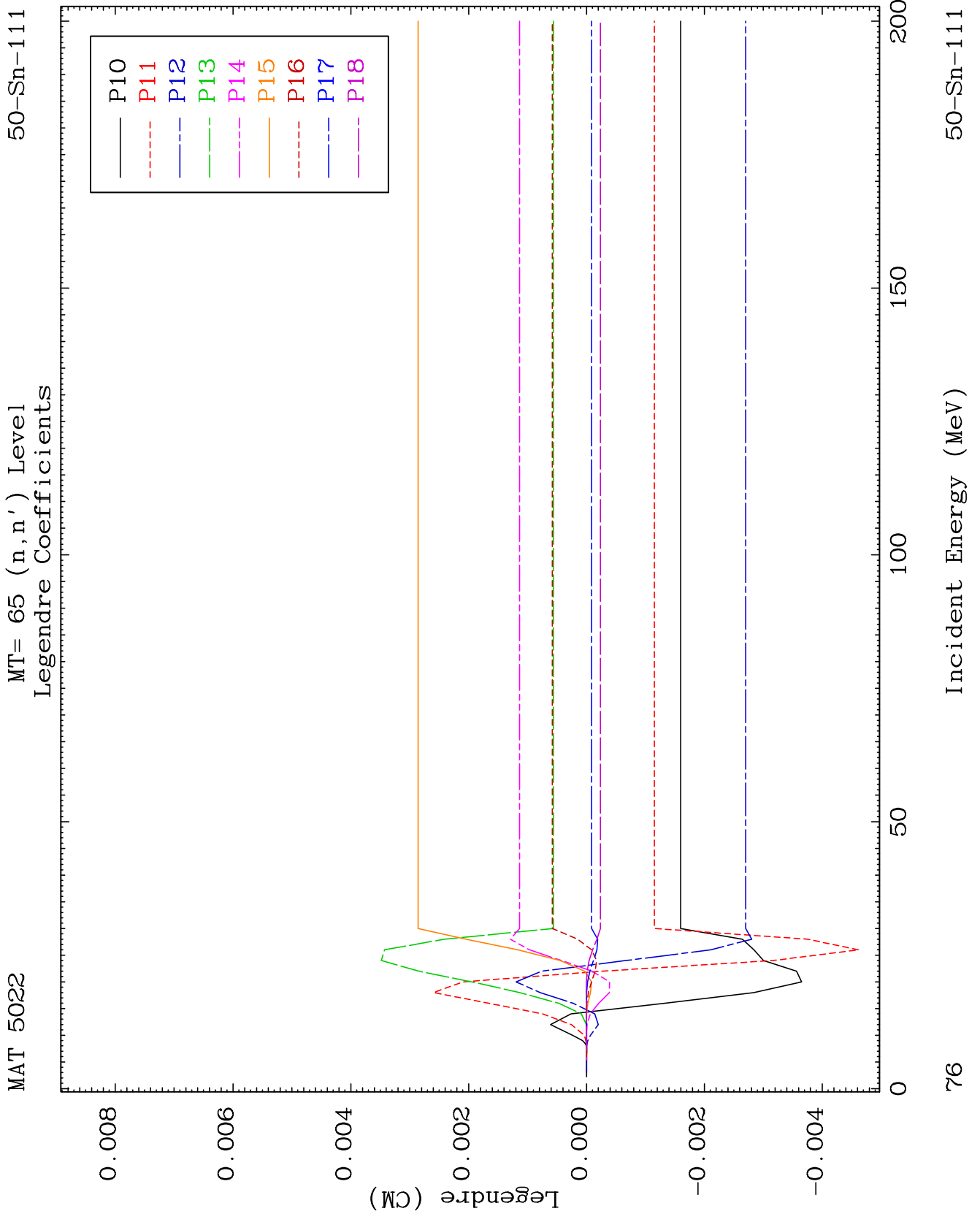
MAT 5022

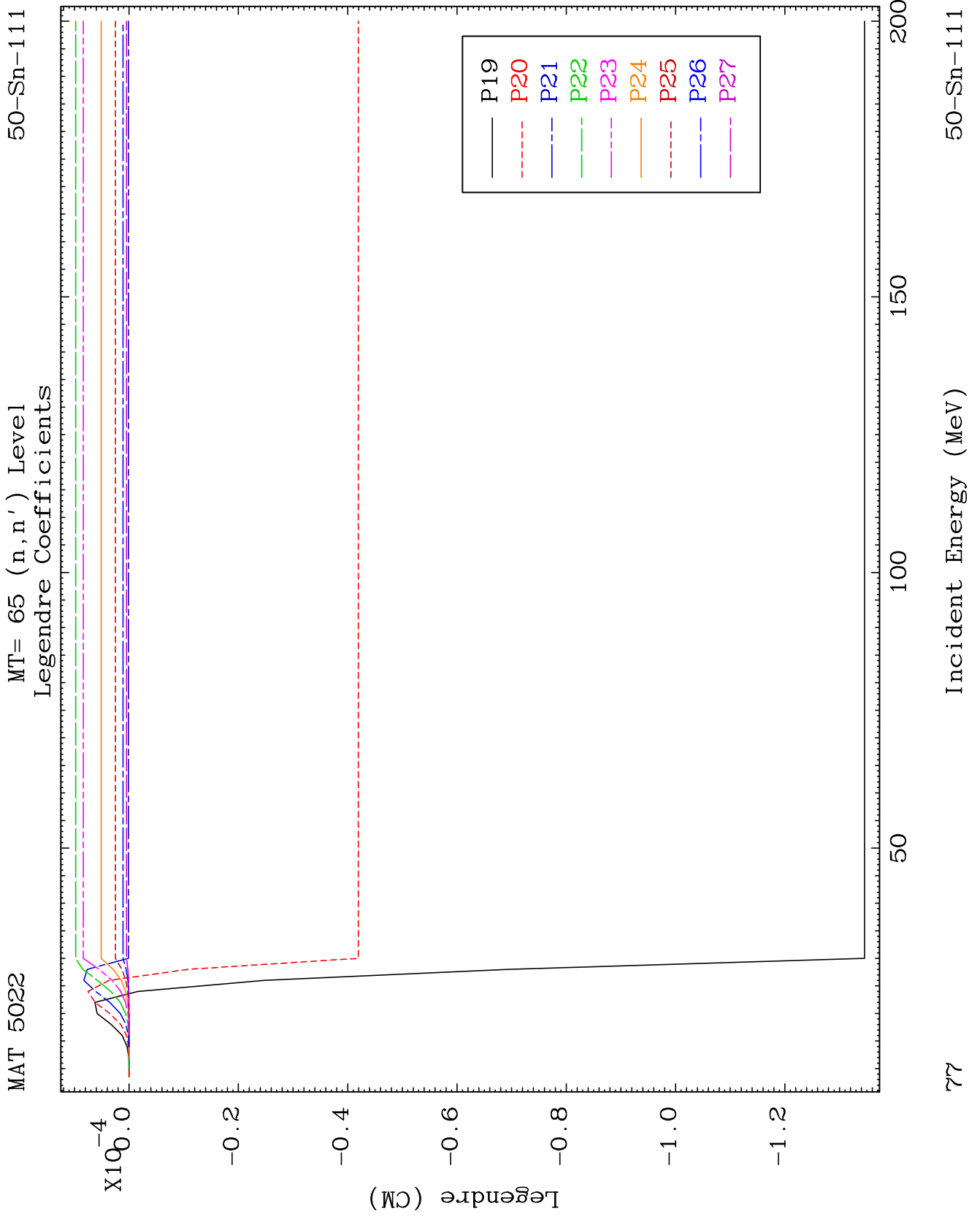
MT= 64 (n,n') Level  
Legendre Coefficients

50-Sn-111





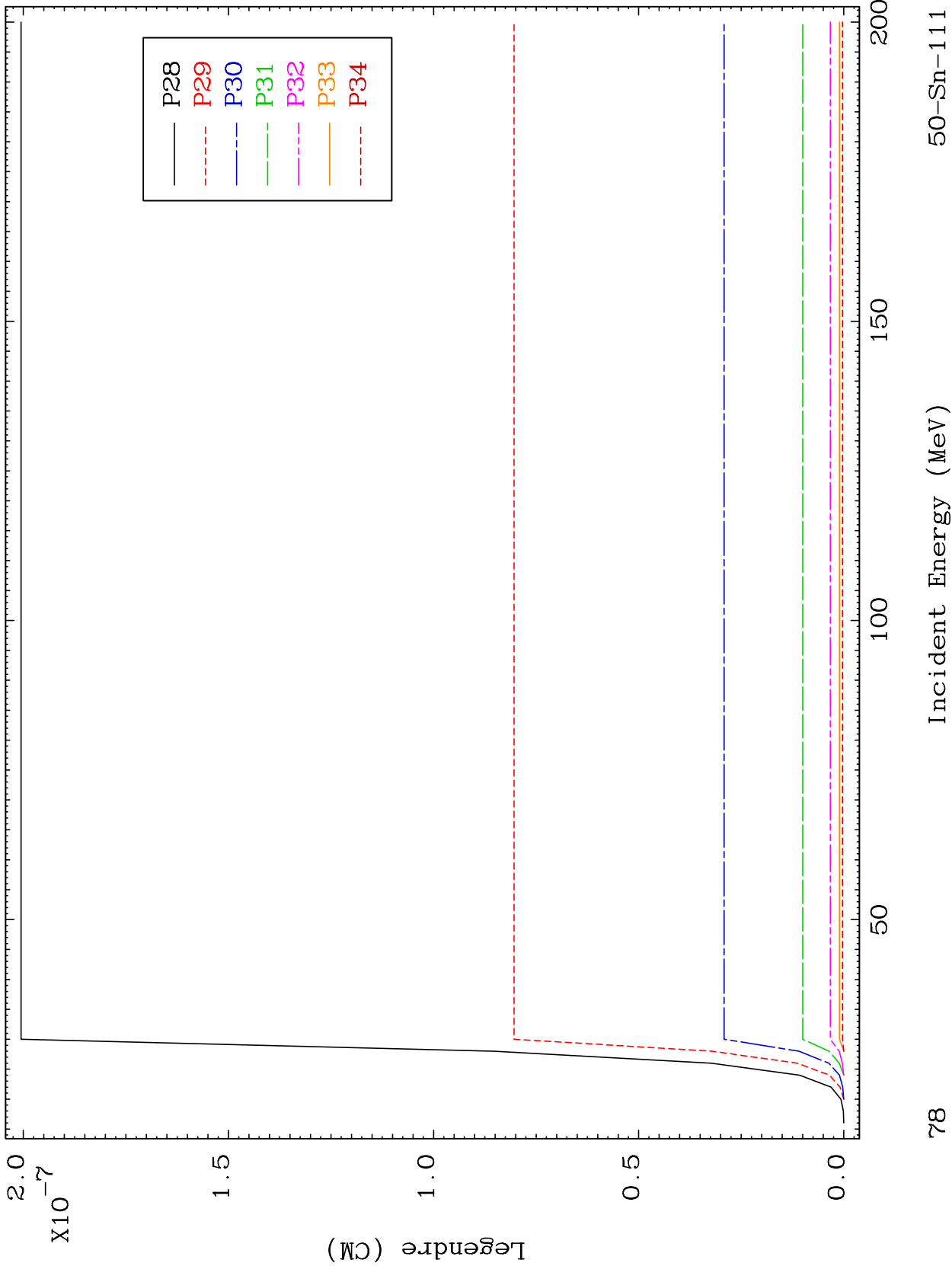




MAT 5022

MT= 65 (n,n') Level  
Legendre Coefficients

50-Sn-111



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Incident Energy (MeV)

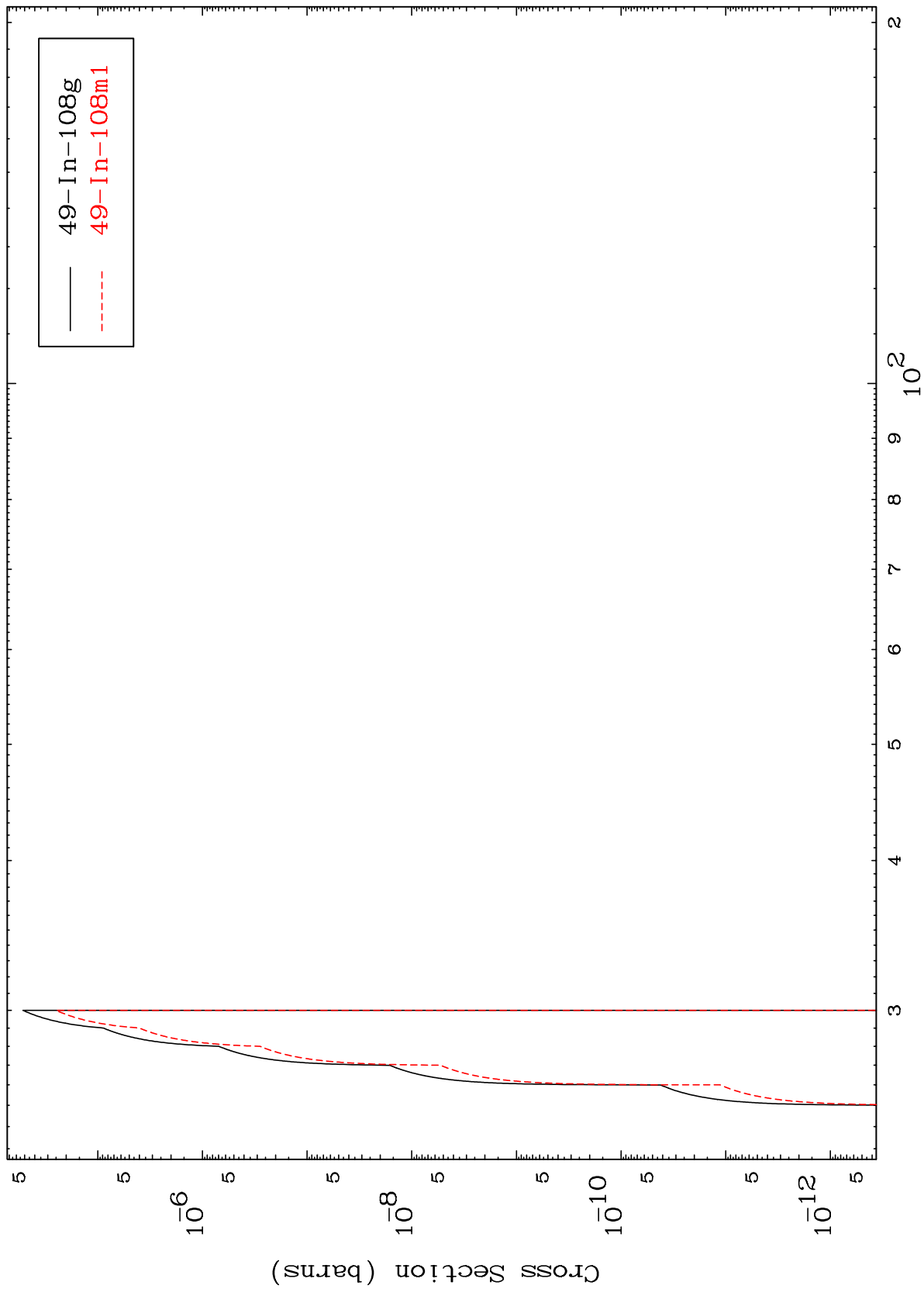
50-Sn-111

MAT 5022

(n,2n) d

50-Sn-111

Radionuclide Production Cross Section



79

Incident Energy (MeV)

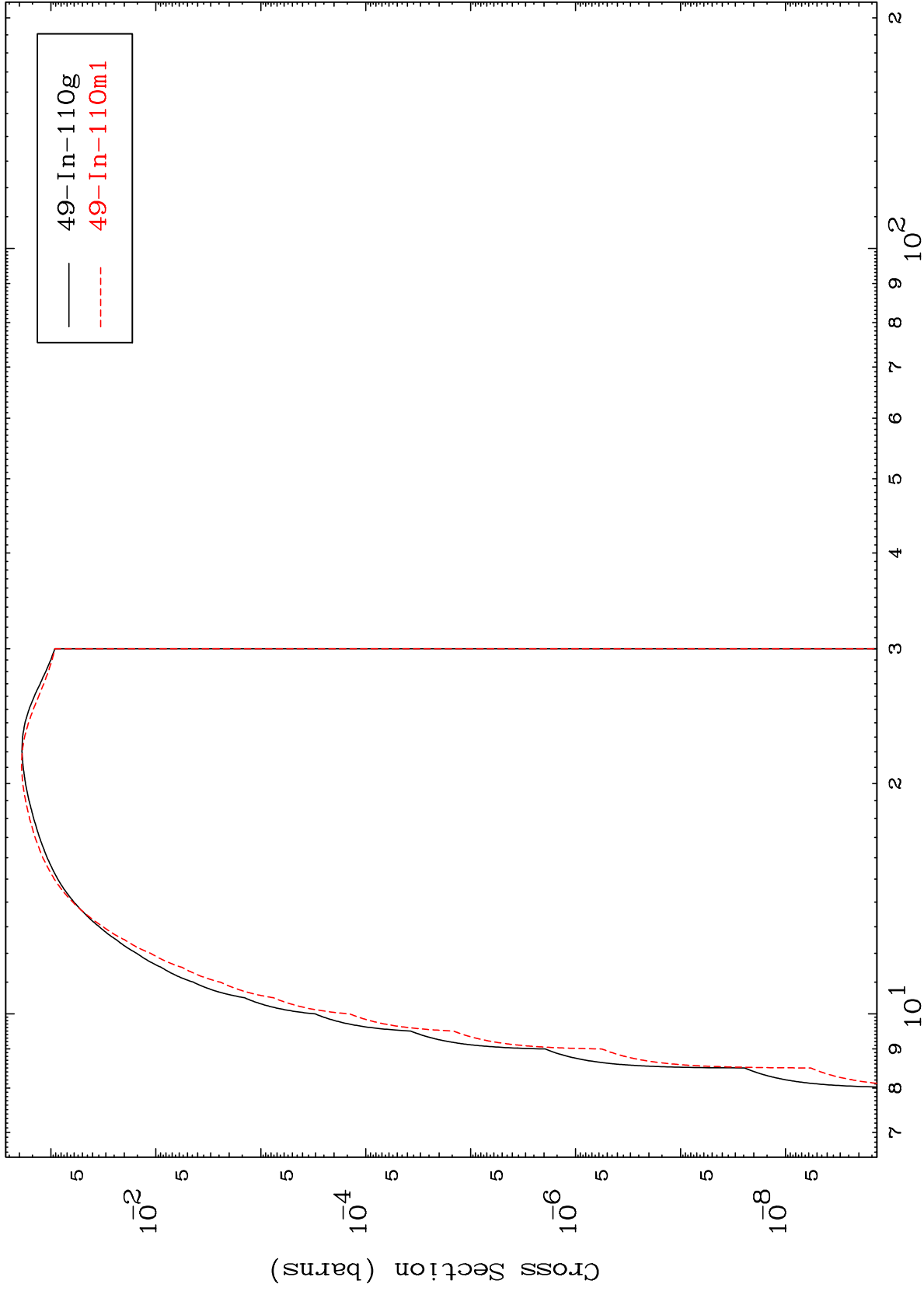
50-Sn-111



MAT 5022

50-Sn-111

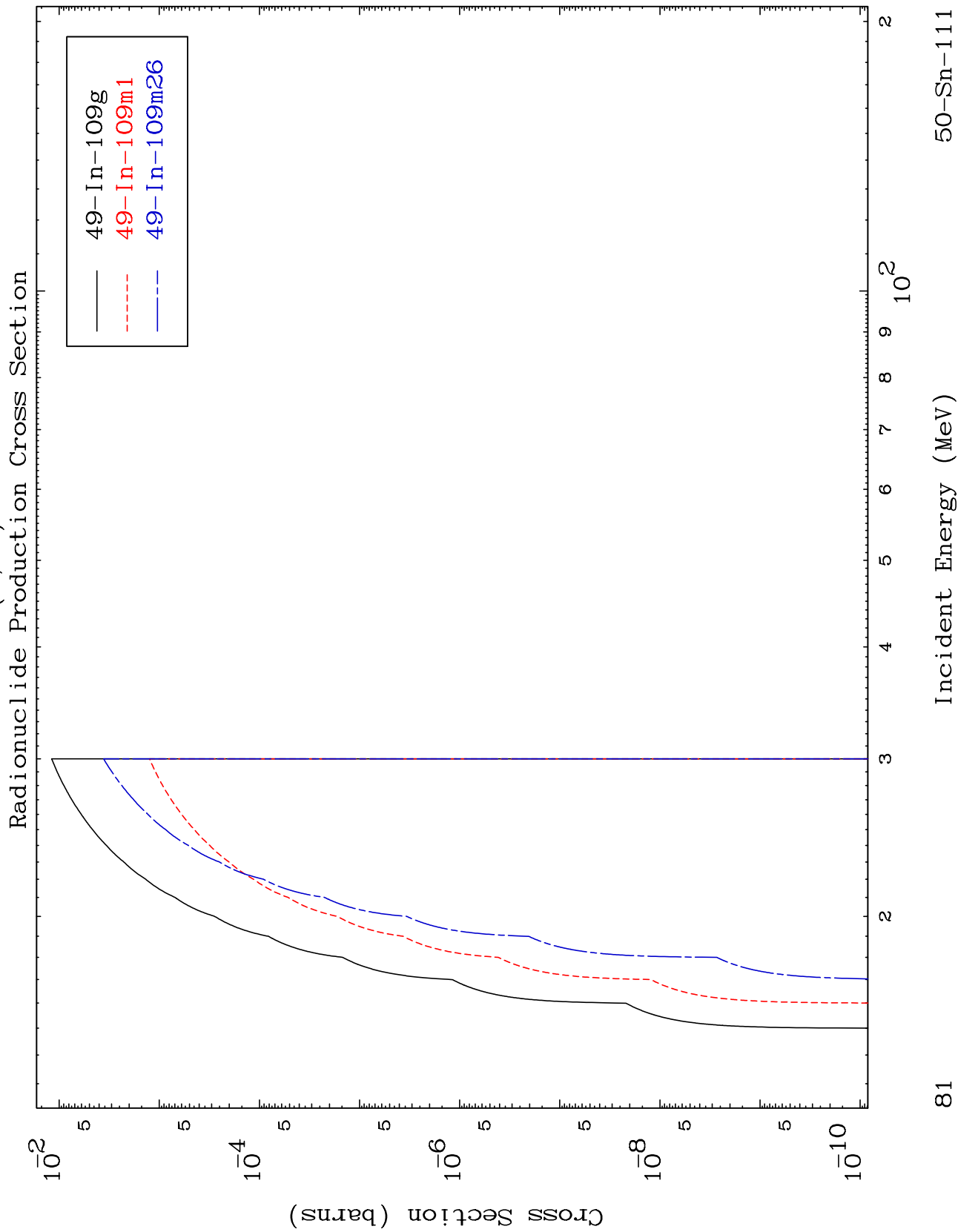
$(n, n')$  p  
Radionuclide Production Cross Section



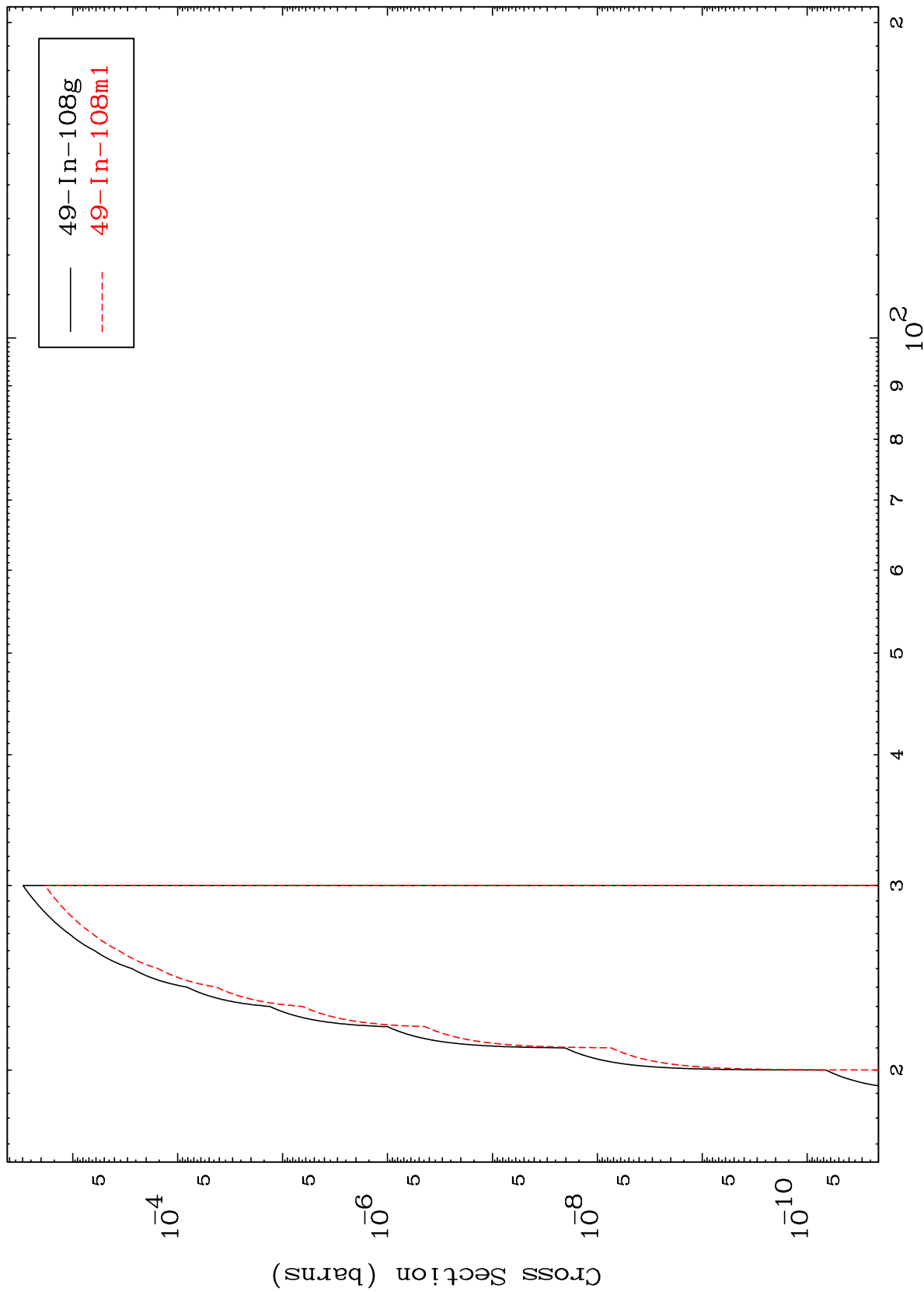
50-Sn-111

Incident Energy (MeV)

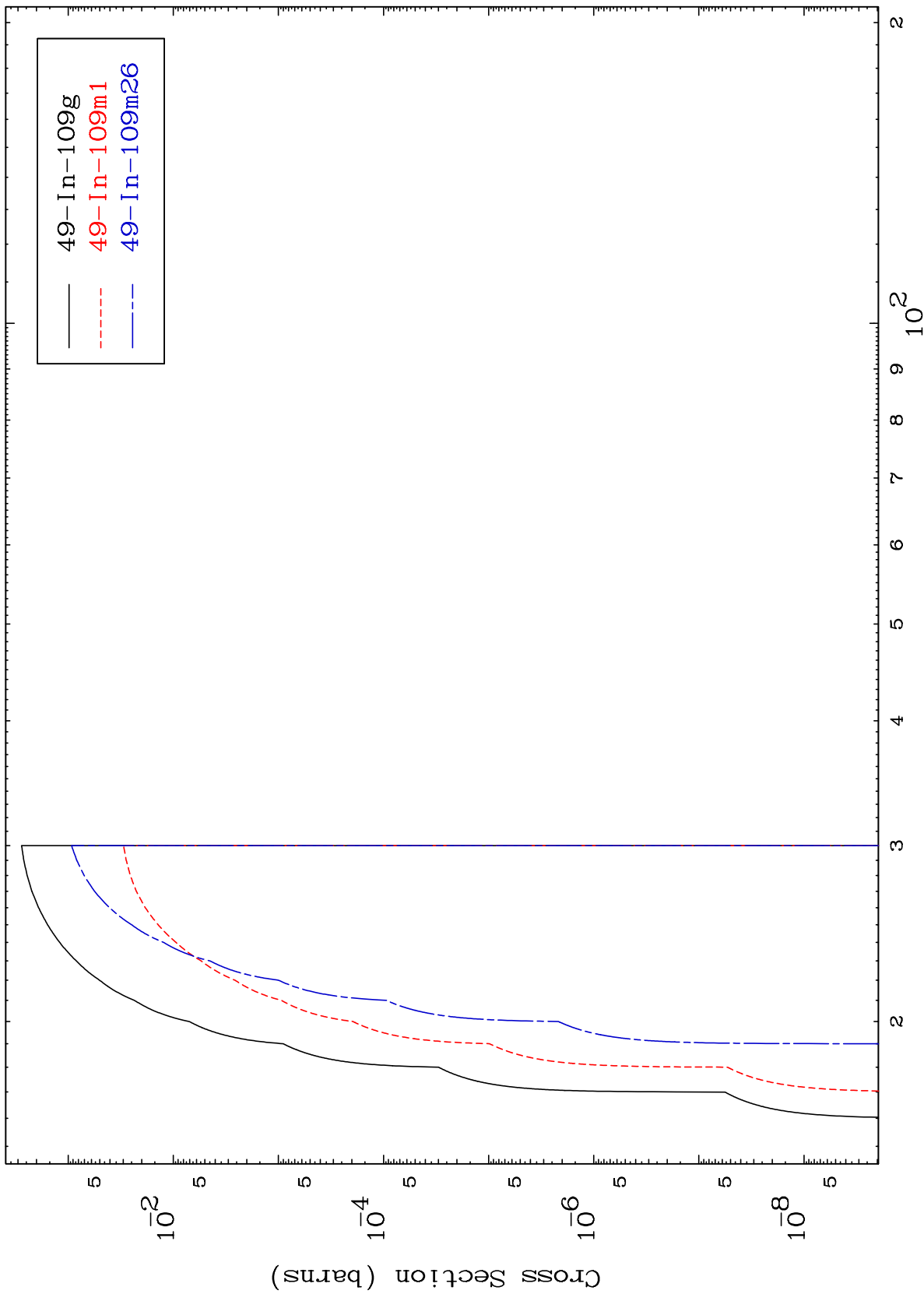
80



Radionuclide Production Cross Section



Radionuclide Production Cross Section

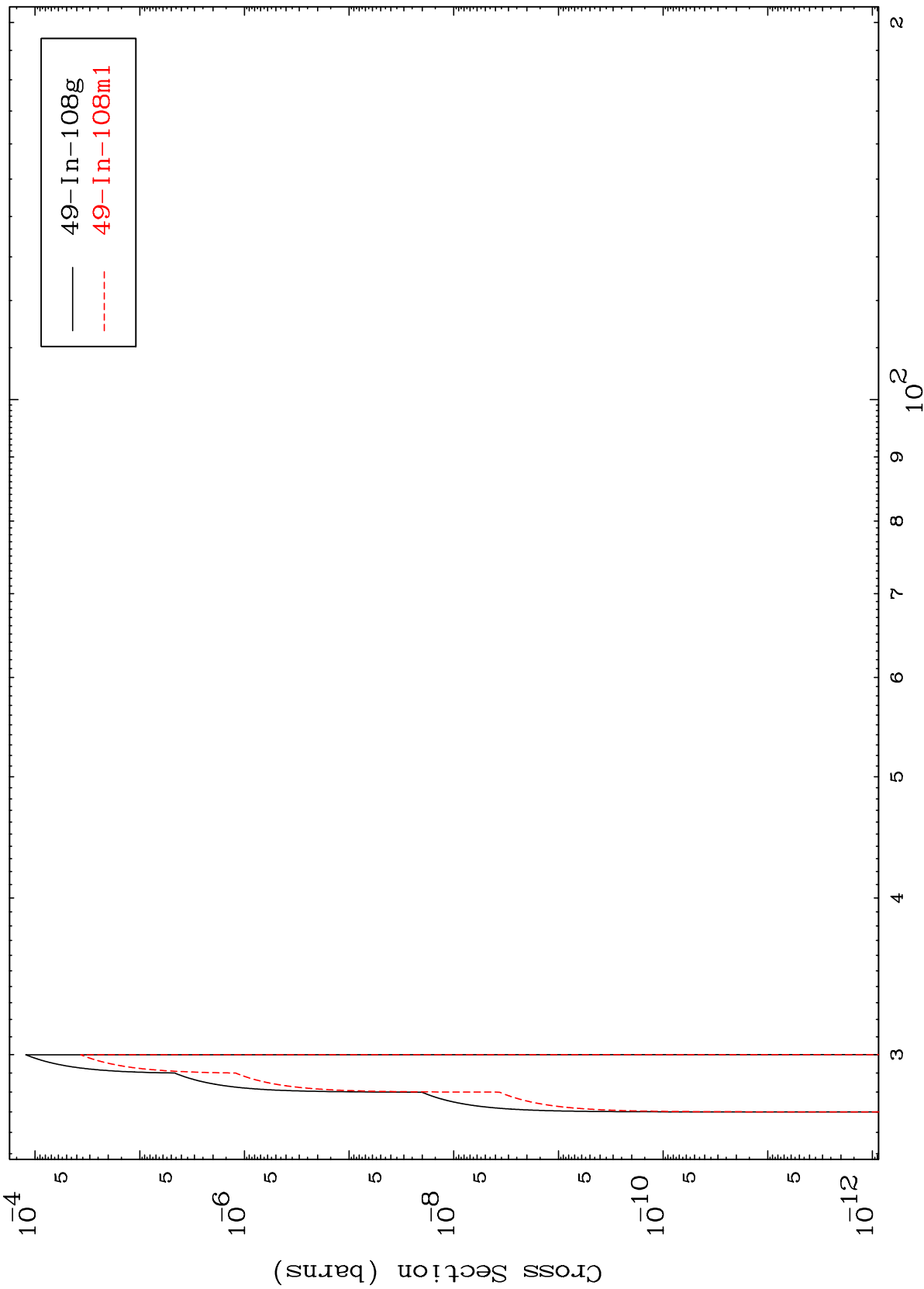


MAT 5022

(n,3n) p

50-Sn-111

Radionuclide Production Cross Section



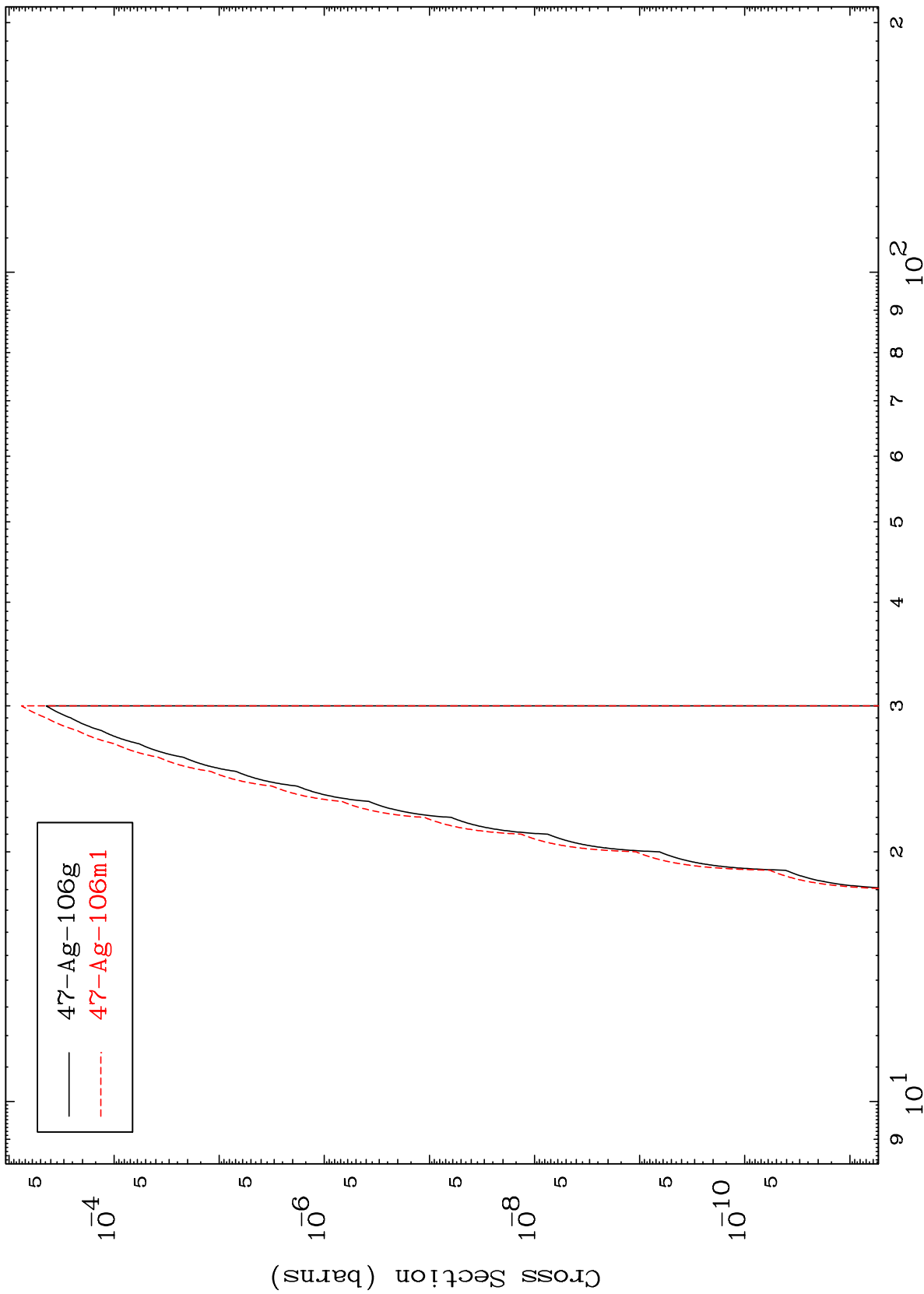
49-In-108g  
49-In-108m1

MAT 5022

(n,n') p  $\alpha$

50-Sn-111

Radionuclide Production Cross Section

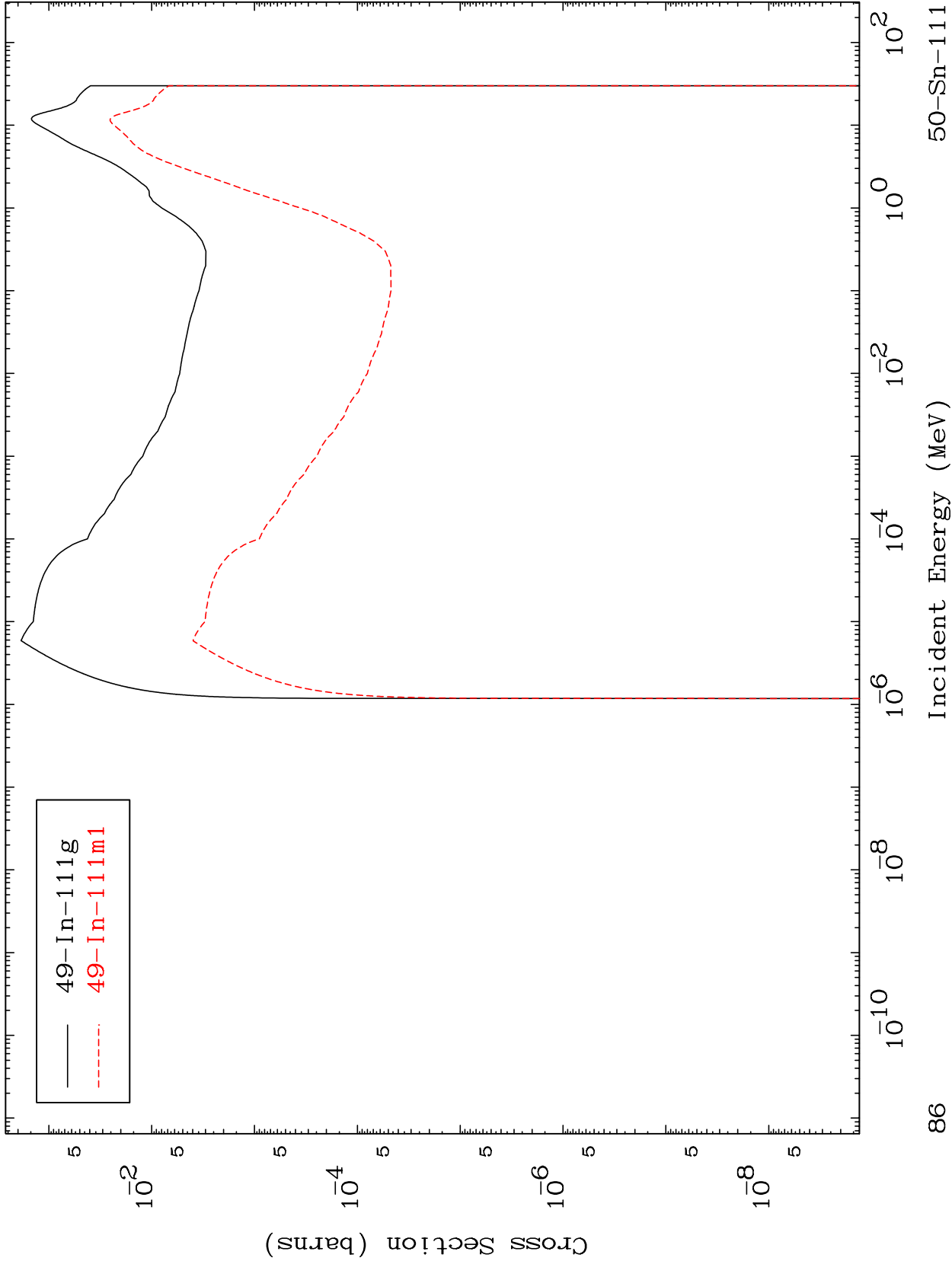


85

Incident Energy (MeV)

50-Sn-111

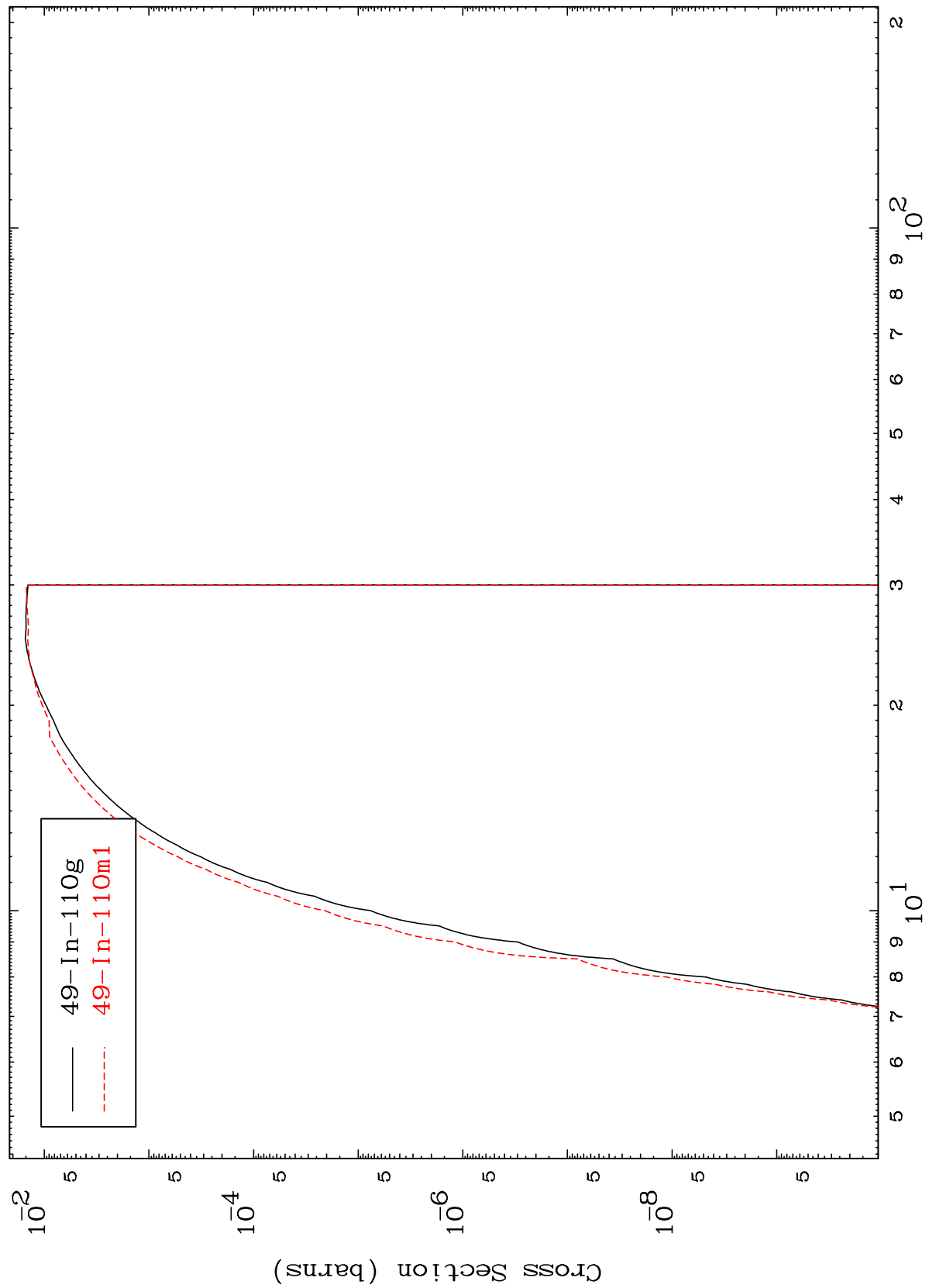
(n,p)  
Radionuclide Production Cross Section



MAT 5022

50-Sn-111

(n,d)  
Radionuclide Production Cross Section

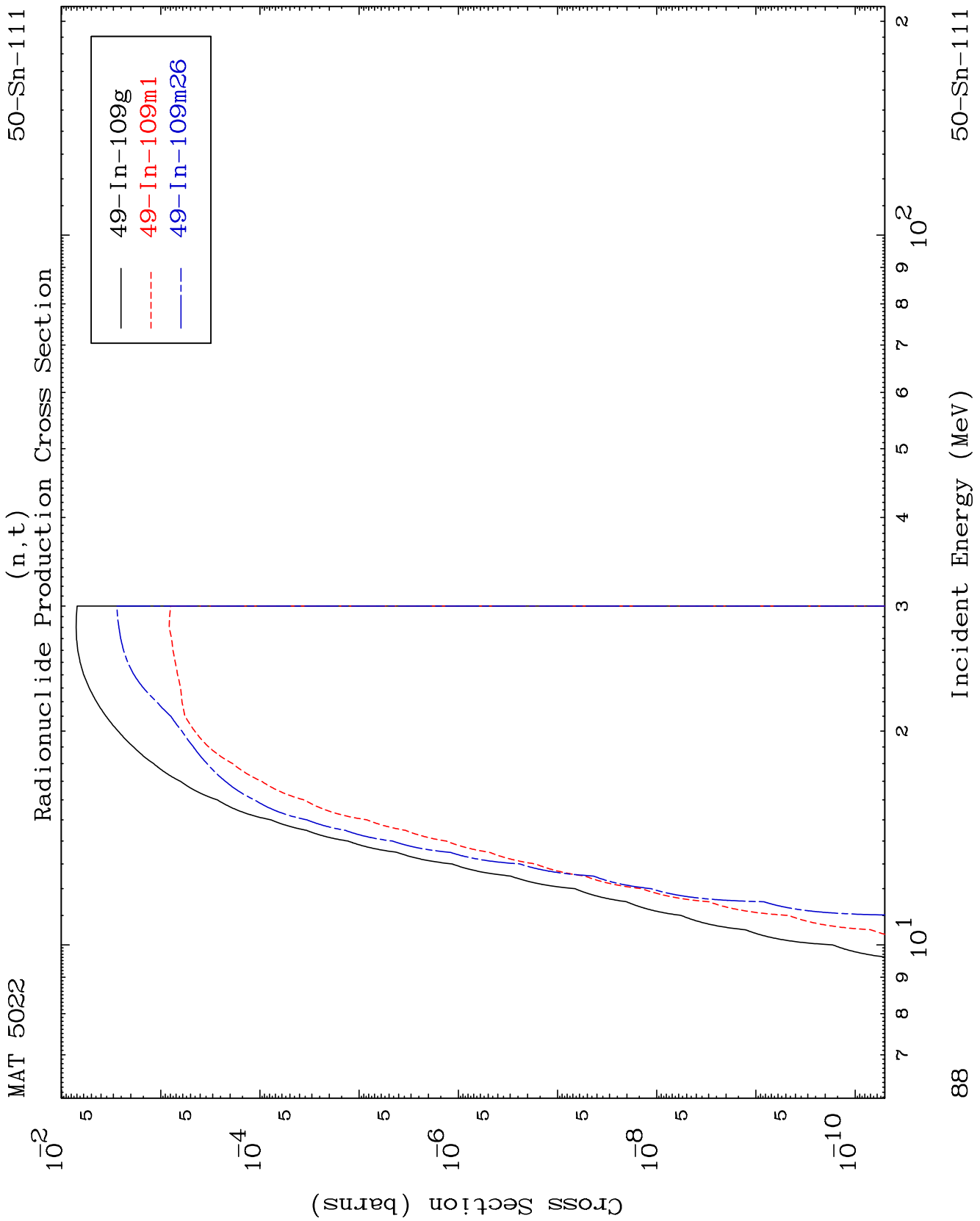


87

Incident Energy (MeV)

50-Sn-111



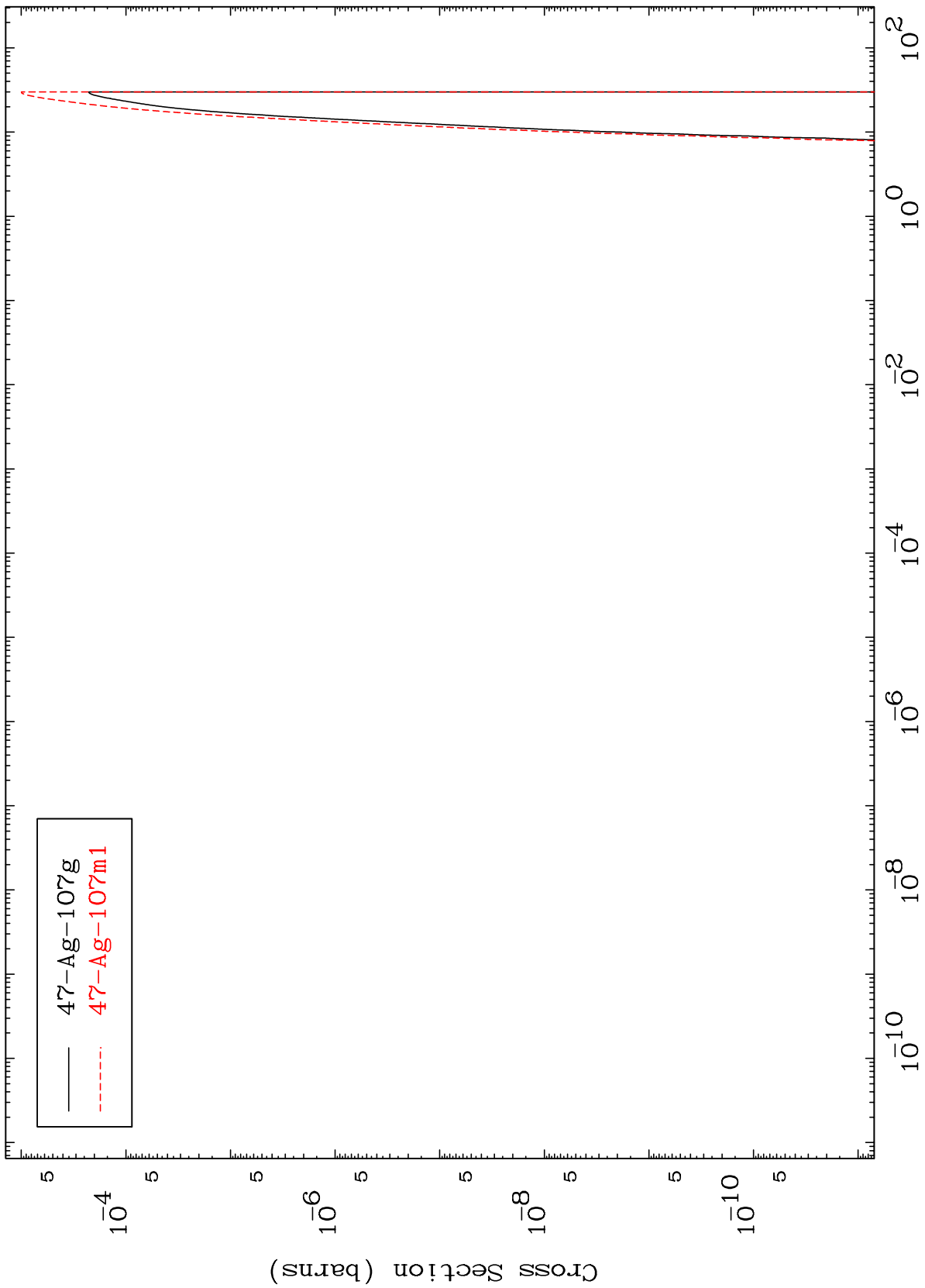


MAT 5022

(n,p)  $\alpha$

50-Sn-111

Radionuclide Production Cross Section

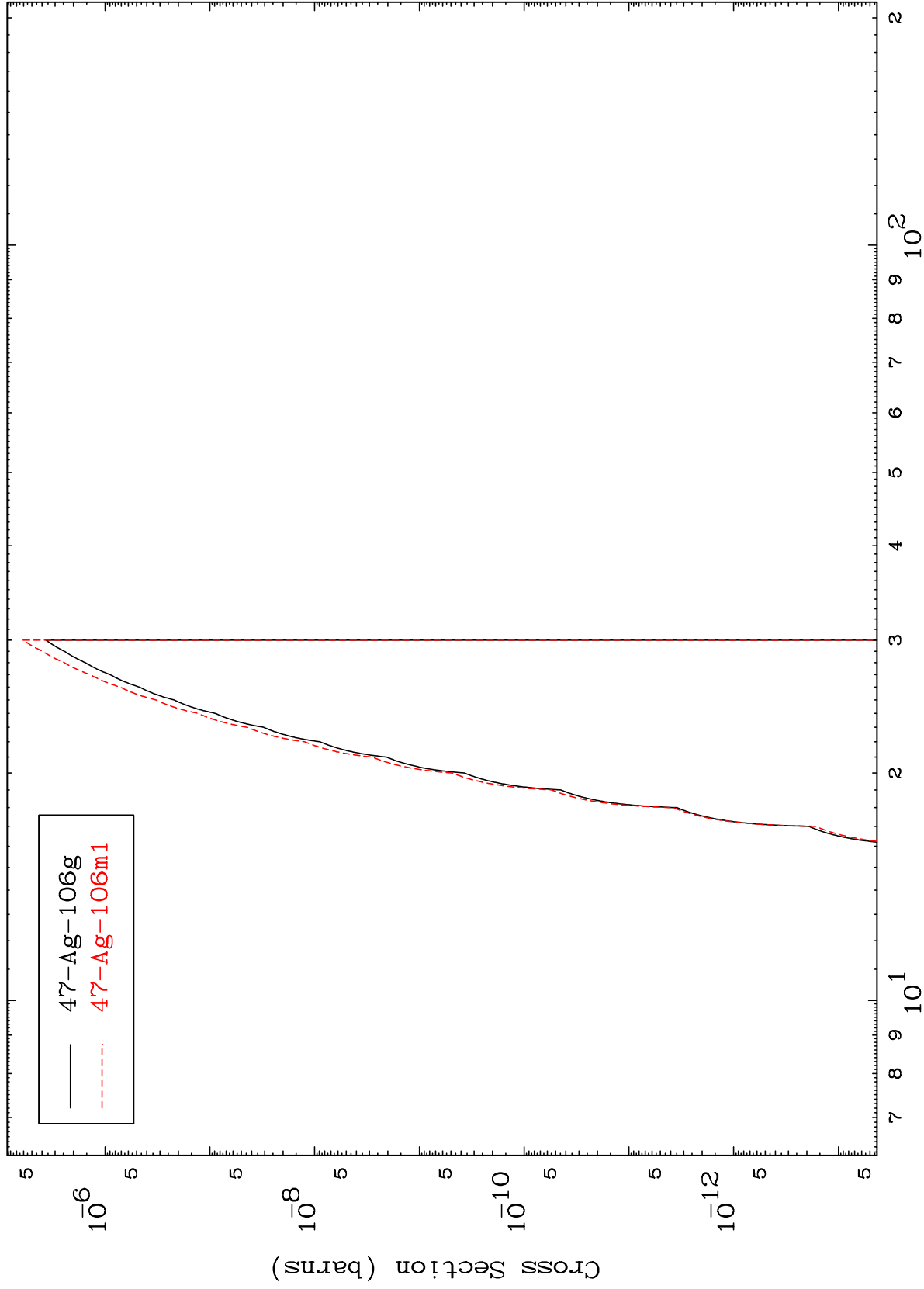


MAT 5022

50-Sn-111

(n,d)  $\alpha$

Radionuclide Production Cross Section



90

Incident Energy (MeV)

50-Sn-111