

Program EVALPLOT
(Version 2021-1)

by

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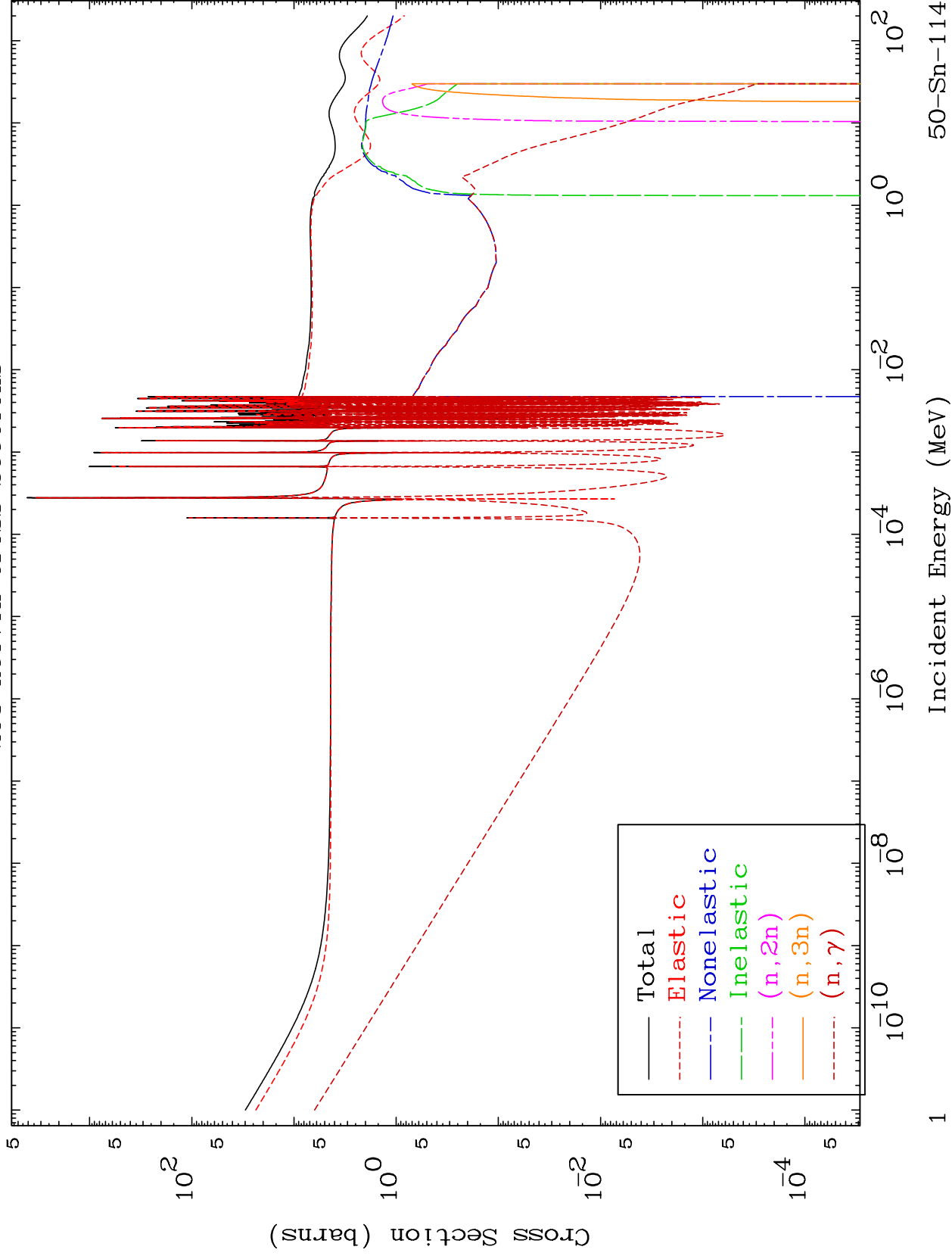
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Press Mouse Button to Start

MAT 5031

Neutron Major
293 Kelvin Cross Sections

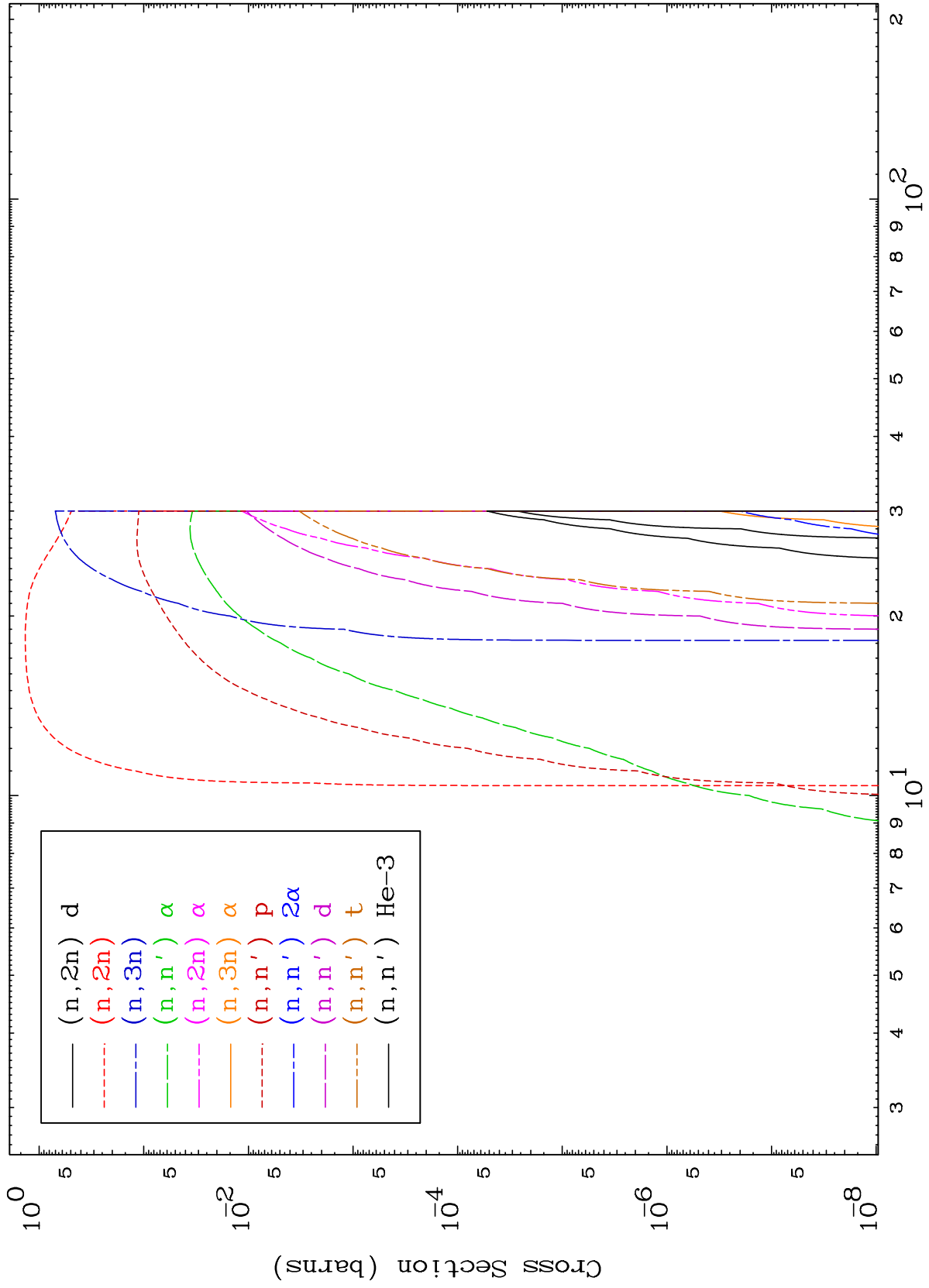
50-Sn-114



MAT 5031

Neutron Absorption
293 Kelvin Cross Sections

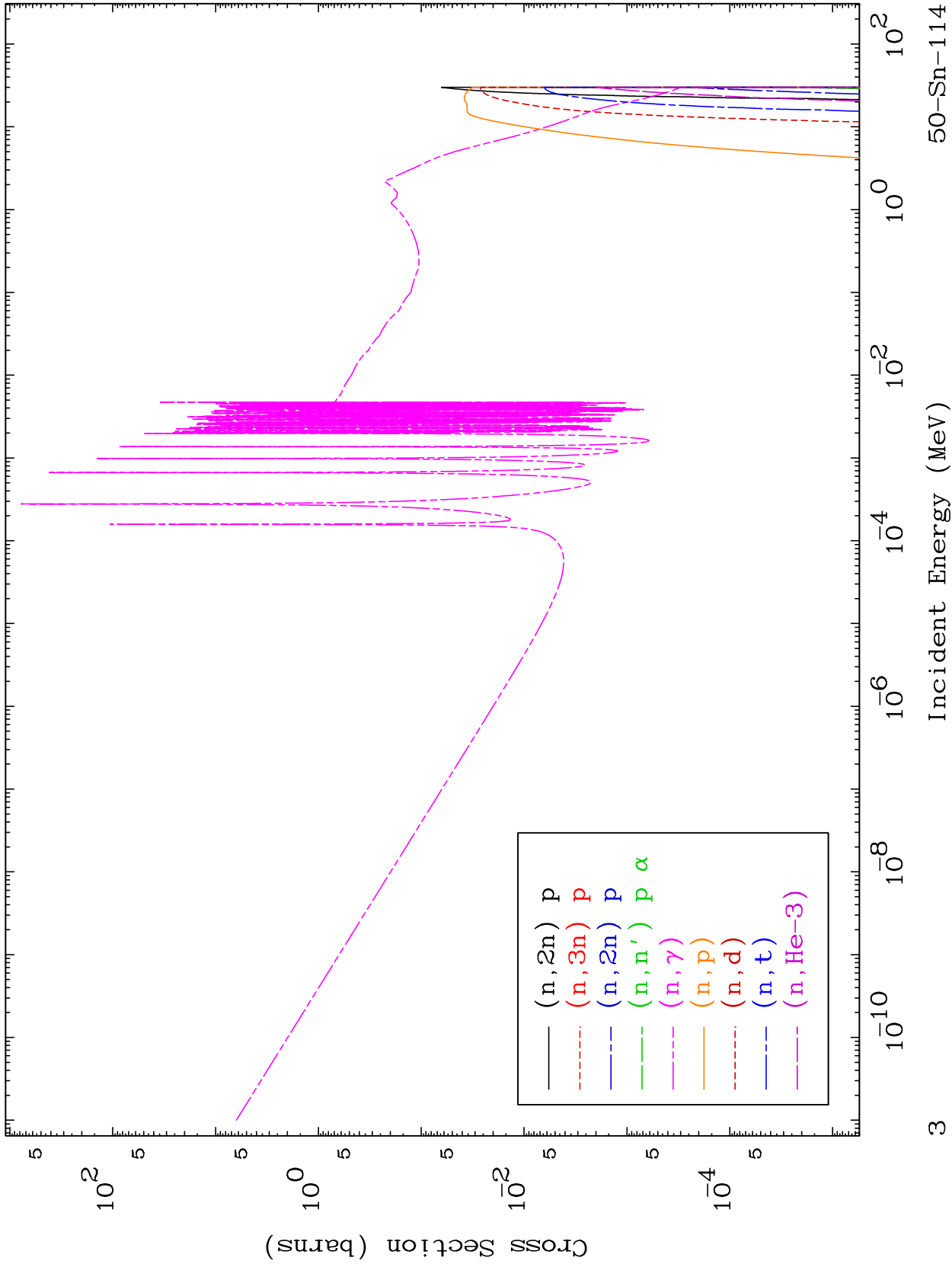
50-Sn-114



MAT 5031

Neutron Absorption
293 Kelvin Cross Sections

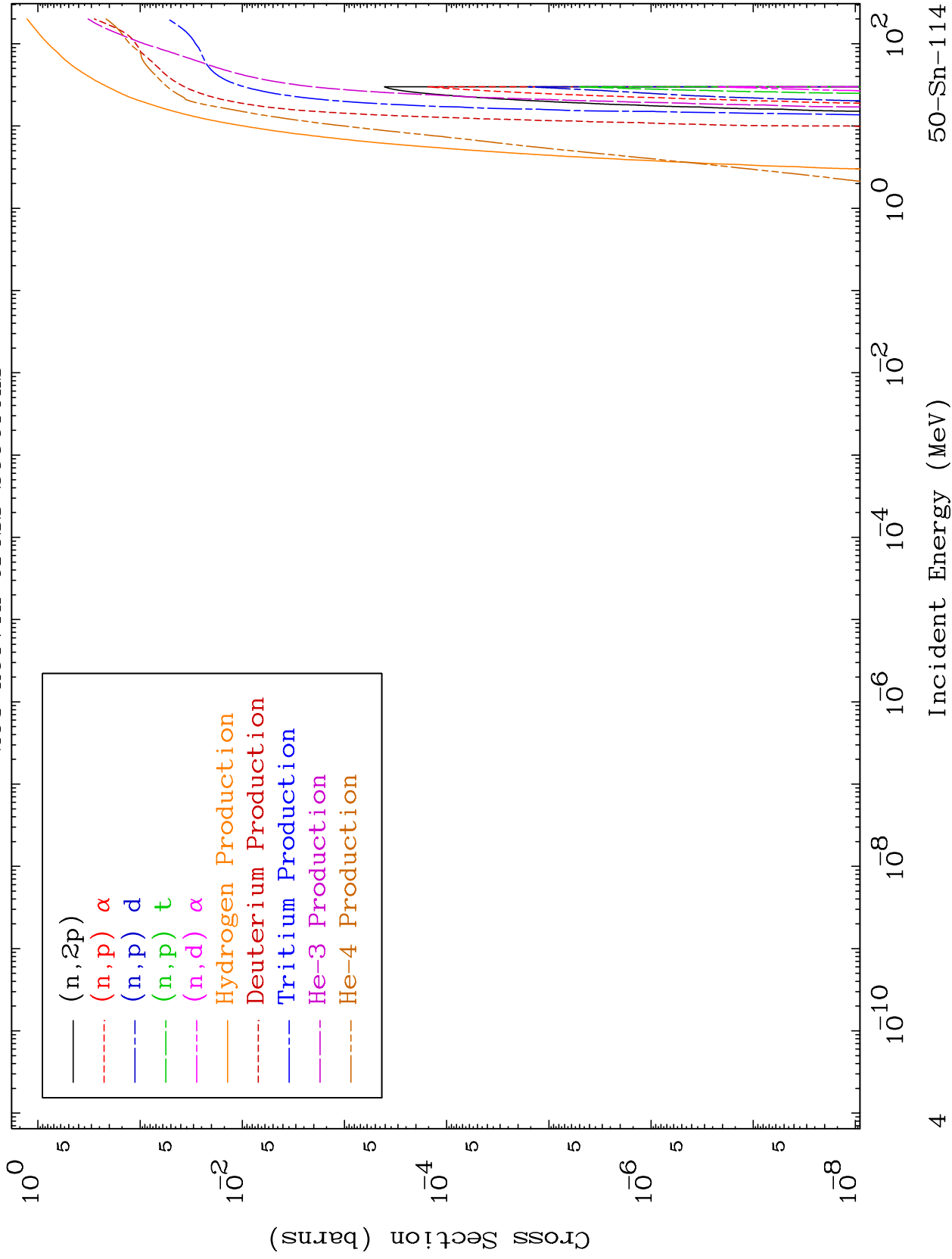
50-Sn-114

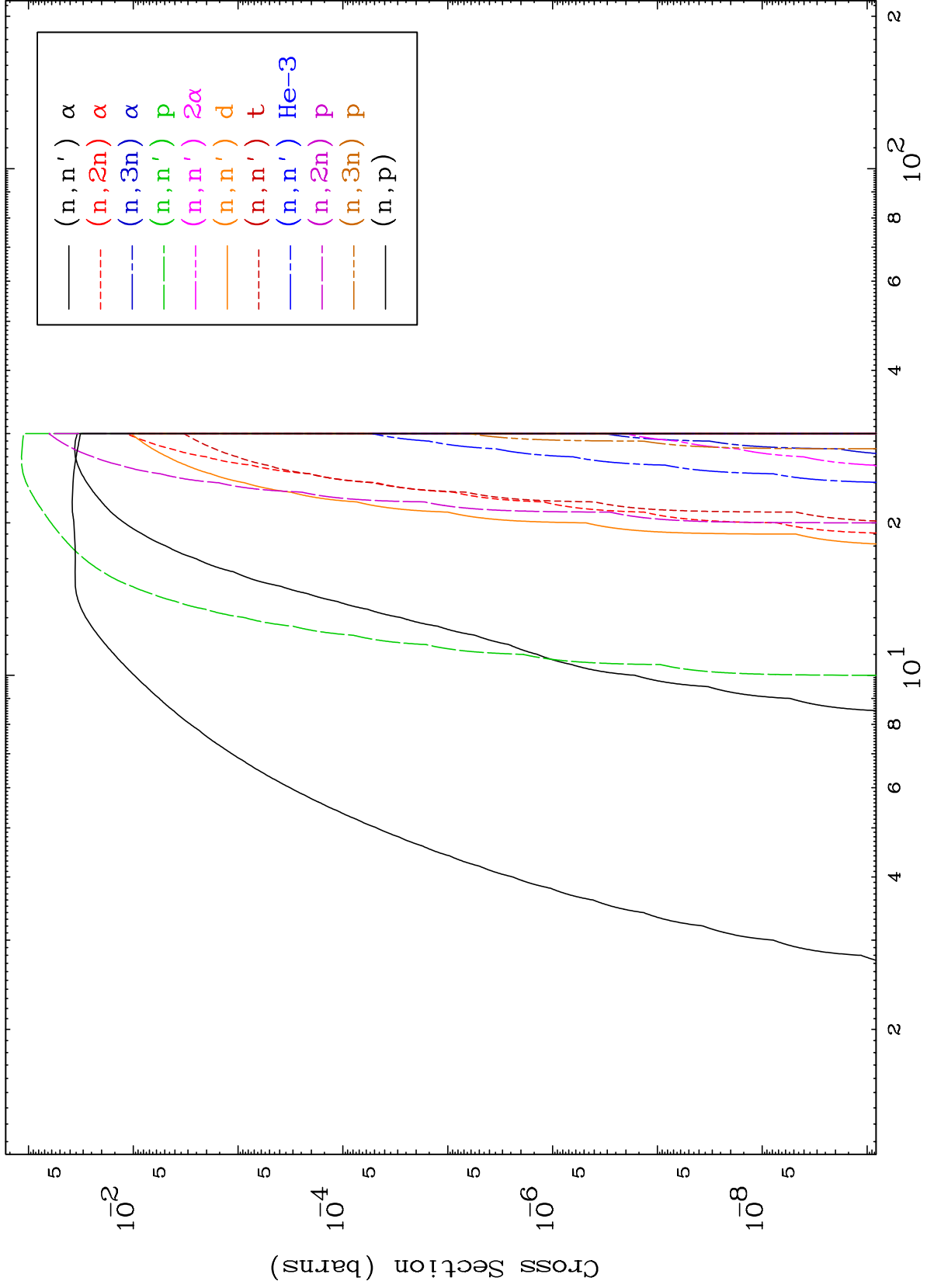


MAT 5031

Neutron Absorption
293 Kelvin Cross Sections

50-Sn-114

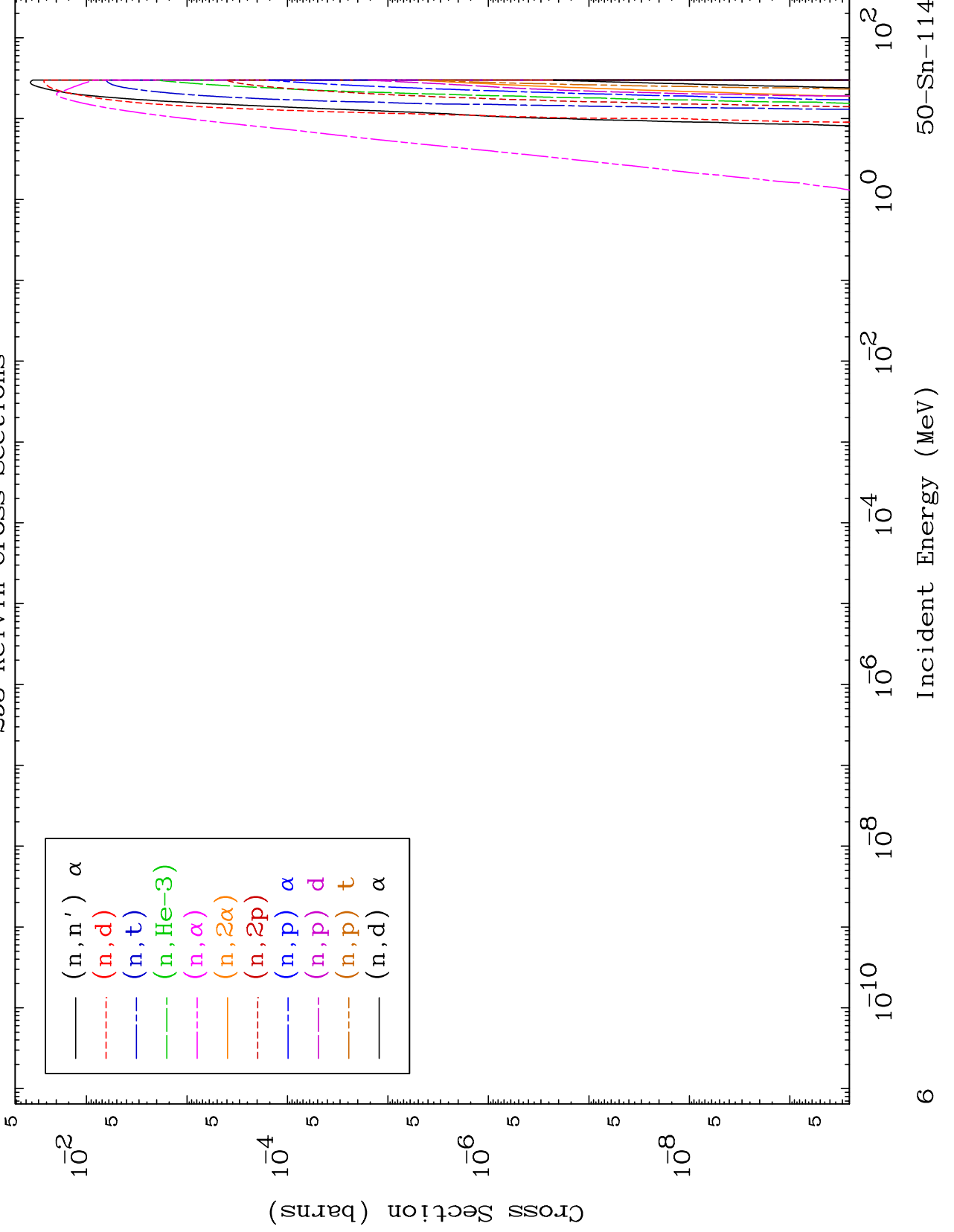




MAT 5031

Charged Particle
293 Kelvin Cross Sections

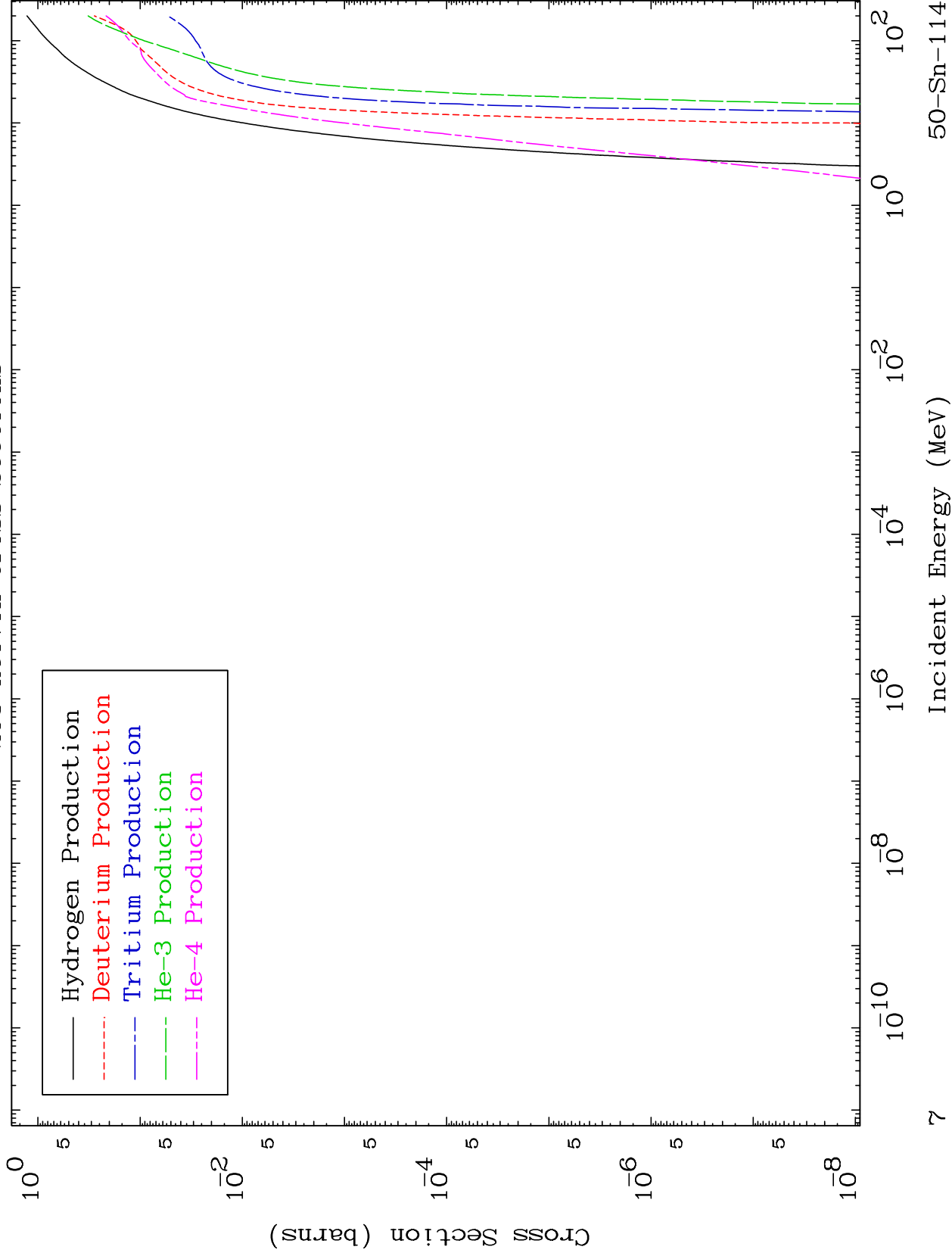
50-Sn-114



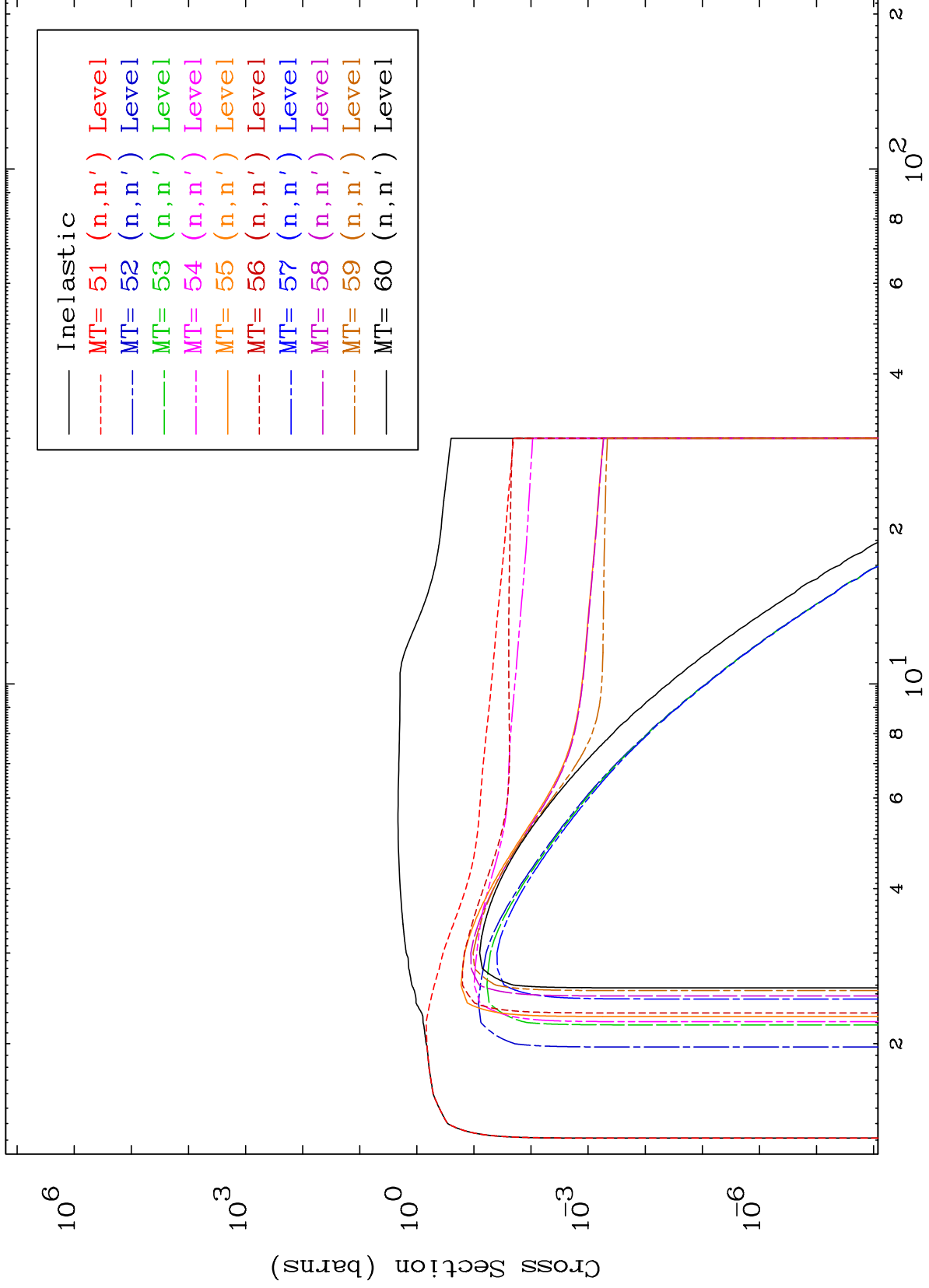
MAT 5031

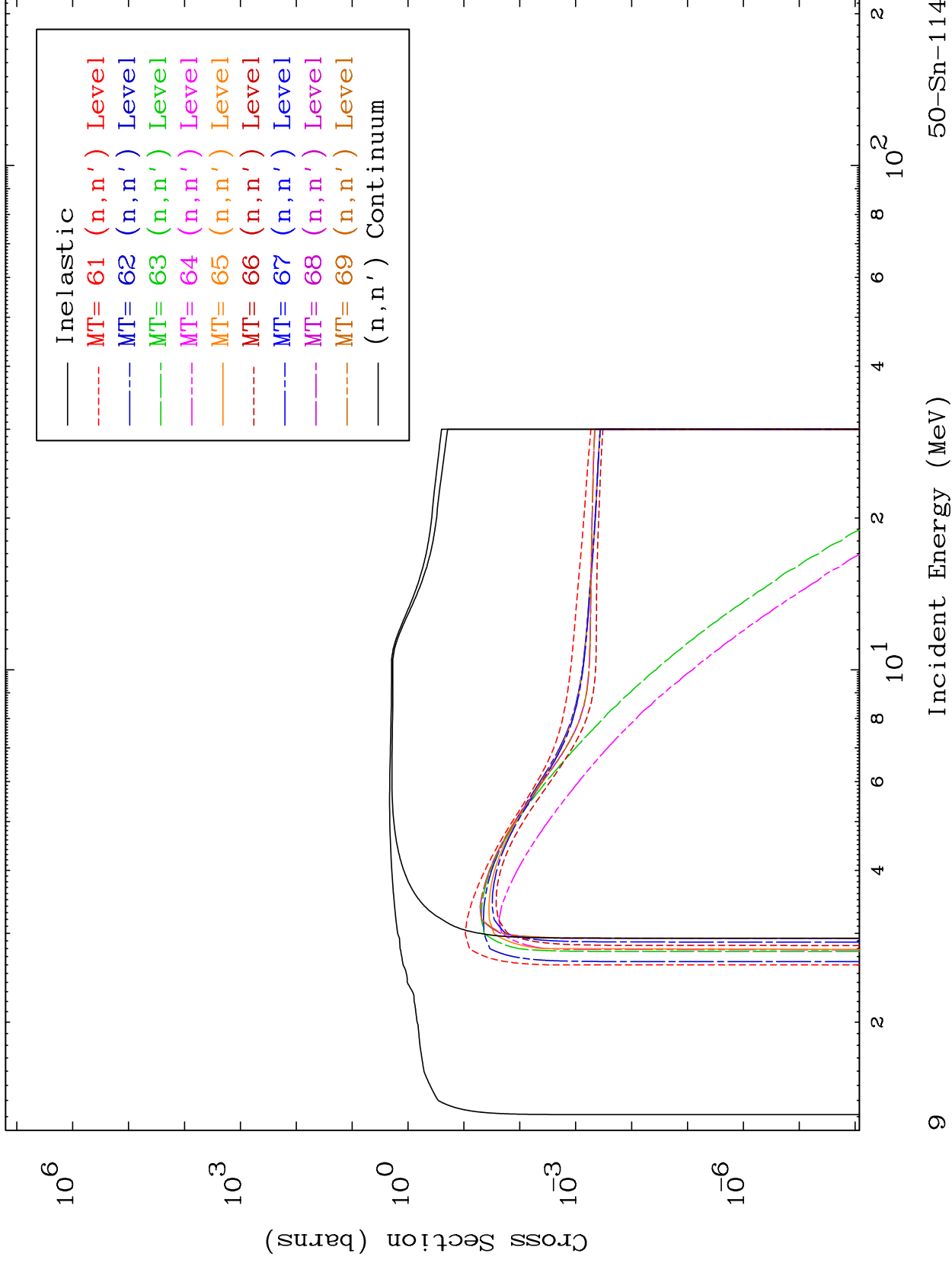
Particle Production
293 Kelvin Cross Sections

50-Sn-114



293 Kelvin Cross Sections

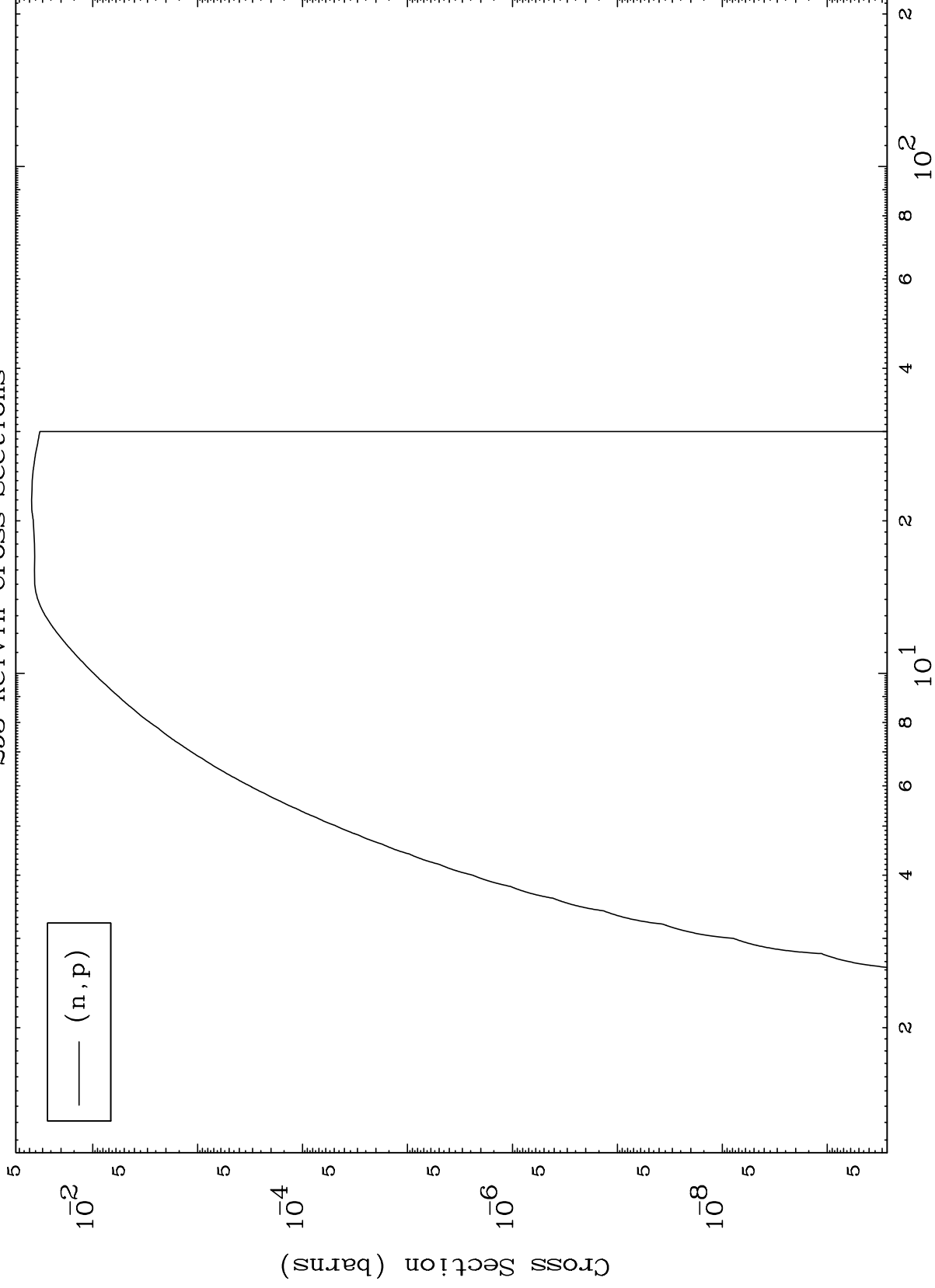




MAT 5031

(n,p) Levels
293 Kelvin Cross Sections

50-Sn-114



10

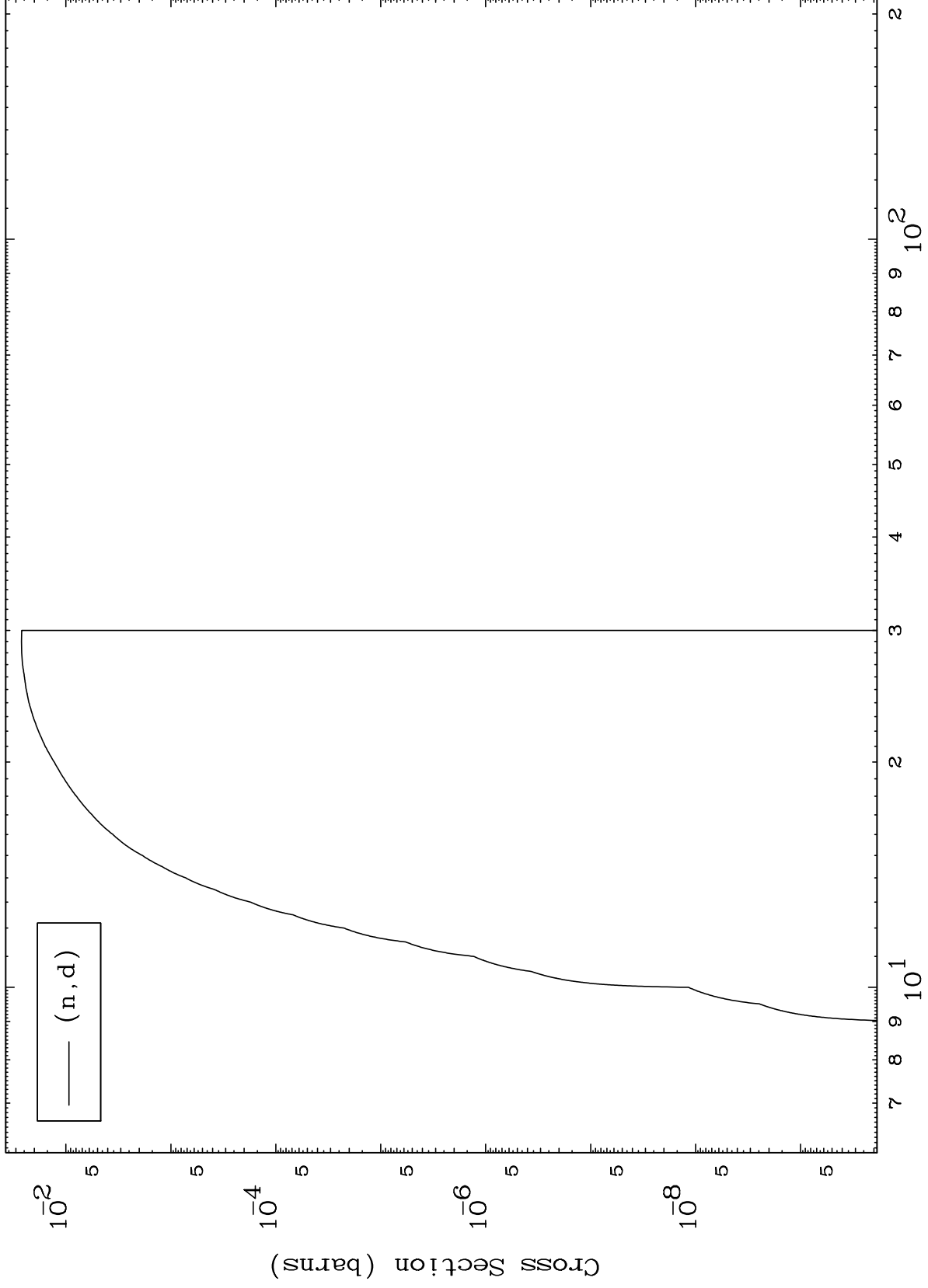
Incident Energy (MeV)

50-Sn-114

MAT 5031

(n,d) Levels
293 Kelvin Cross Sections

50-Sn-114



11

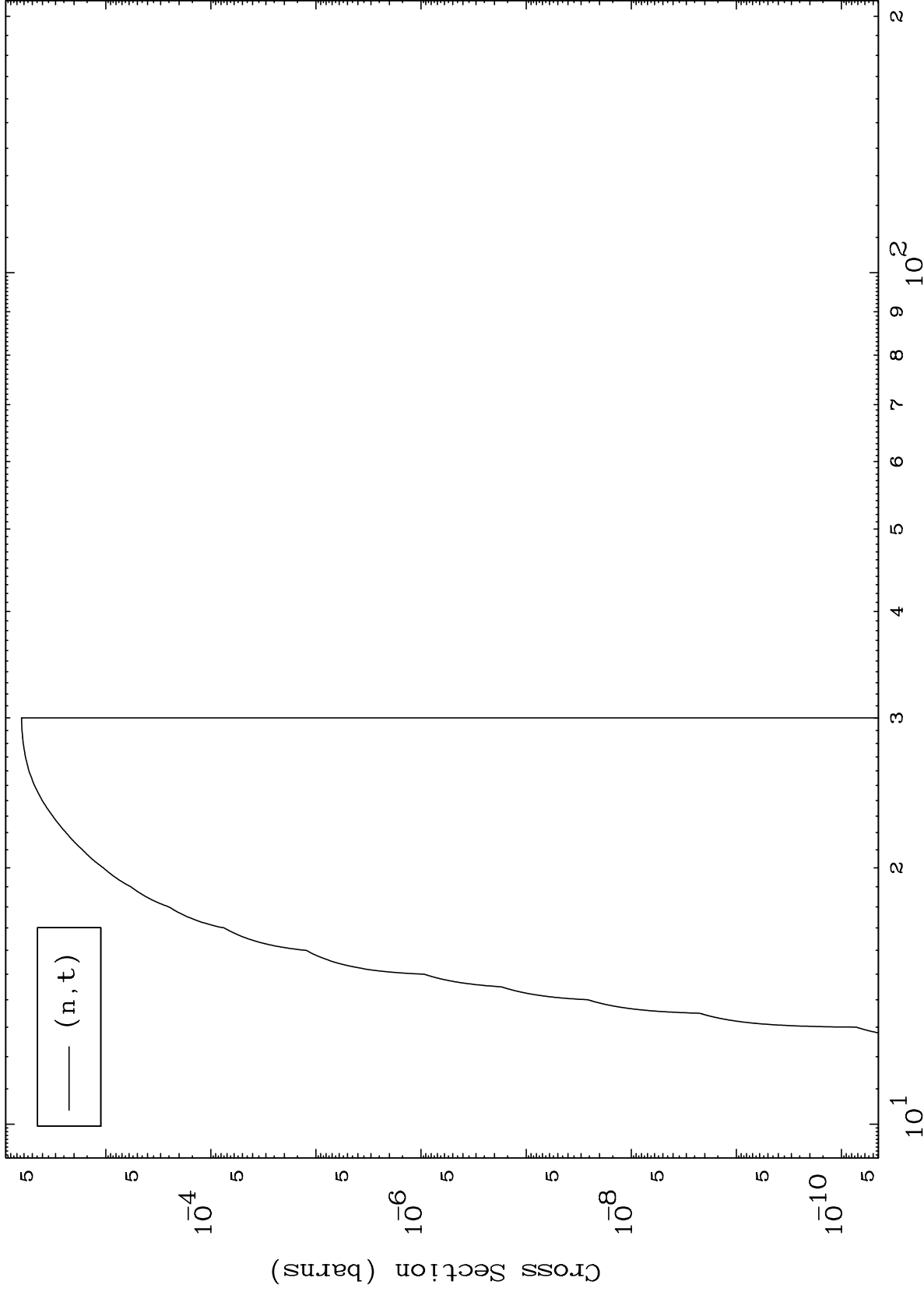
Incident Energy (MeV)

50-Sn-114

MAT 5031

(n,t) Levels
293 Kelvin Cross Sections

50-Sn-114



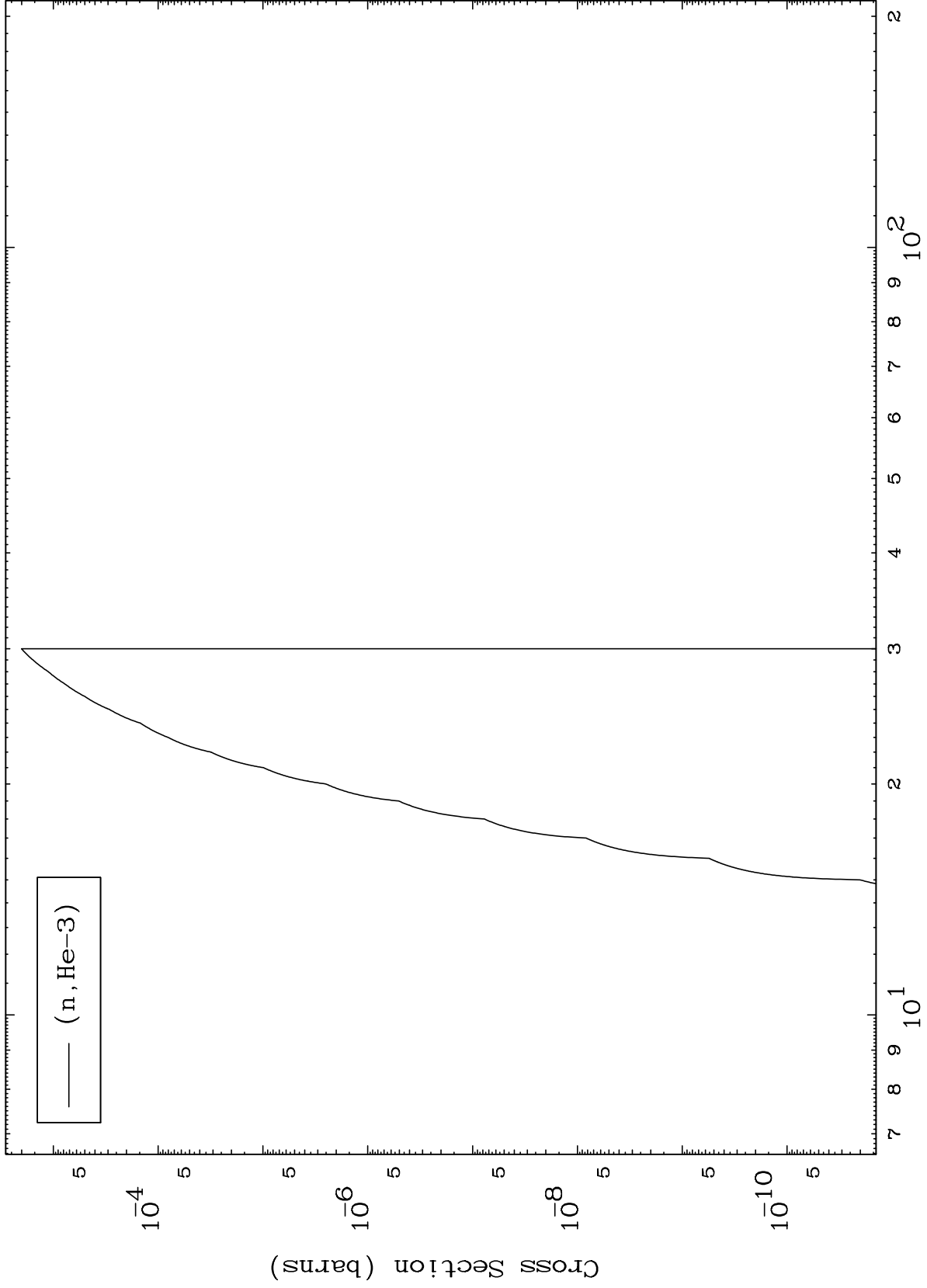
Incident Energy (MeV)

50-Sn-114

MAT 5031

(n,He3) Levels
293 Kelvin Cross Sections

50-Sn-114



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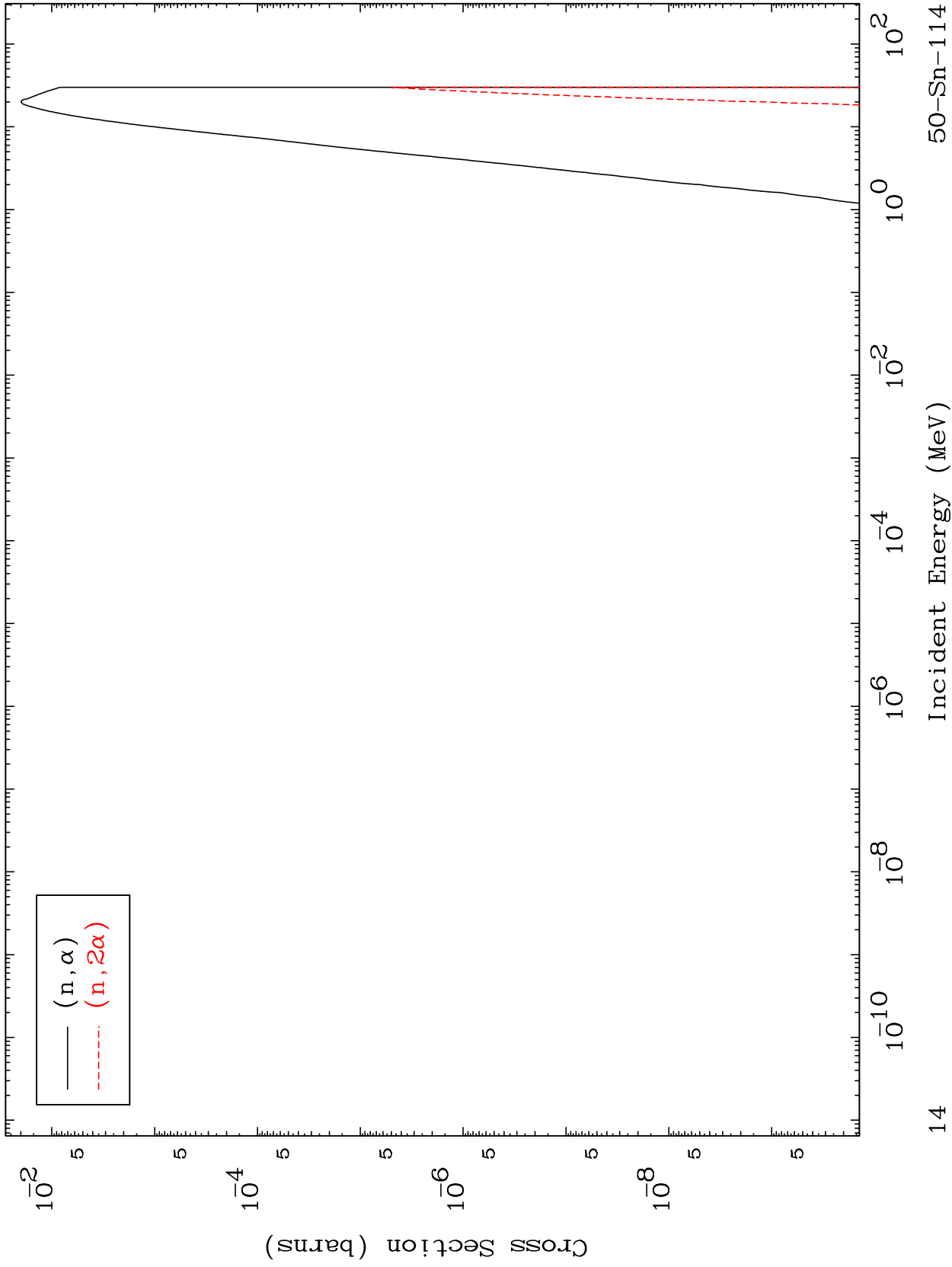
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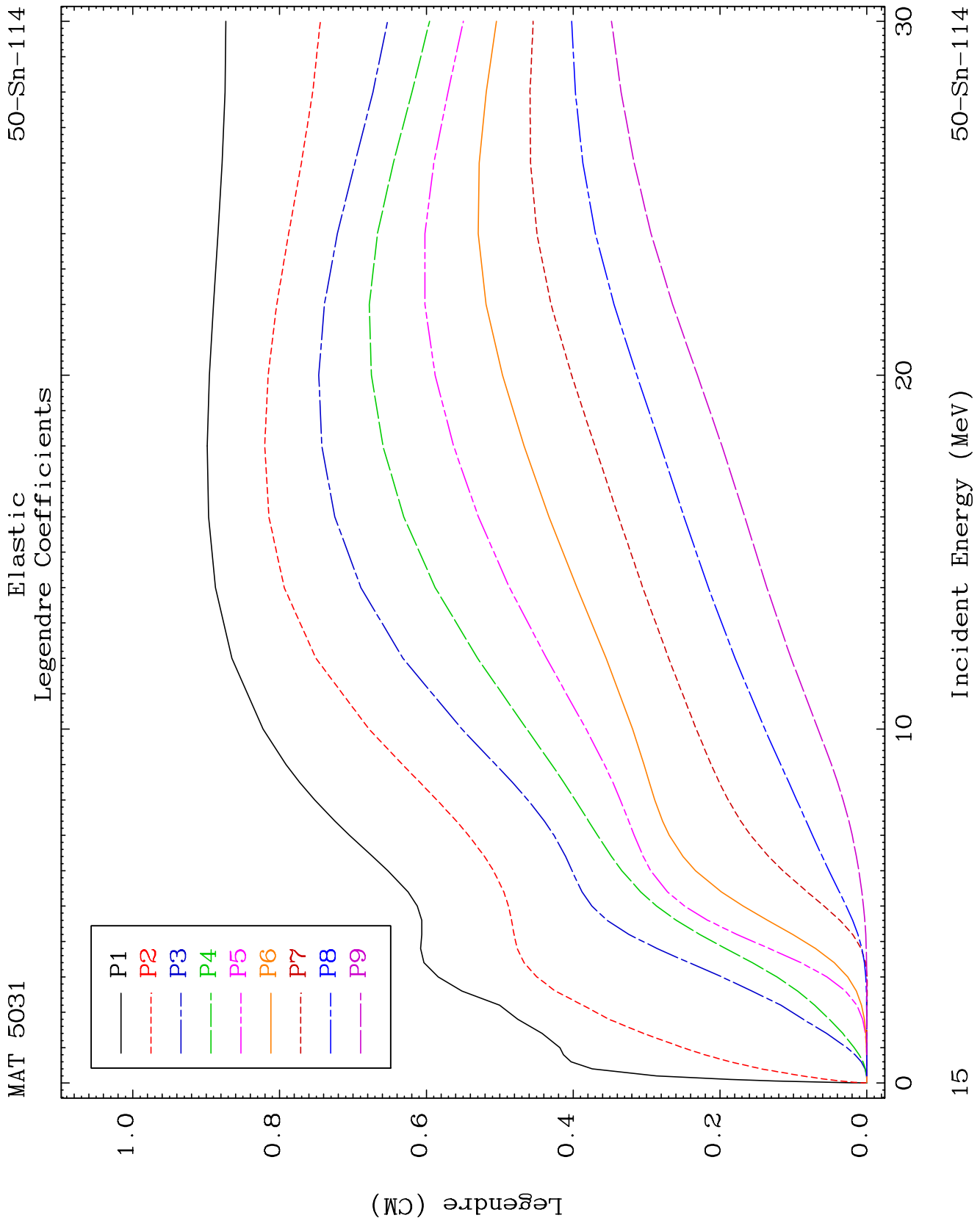
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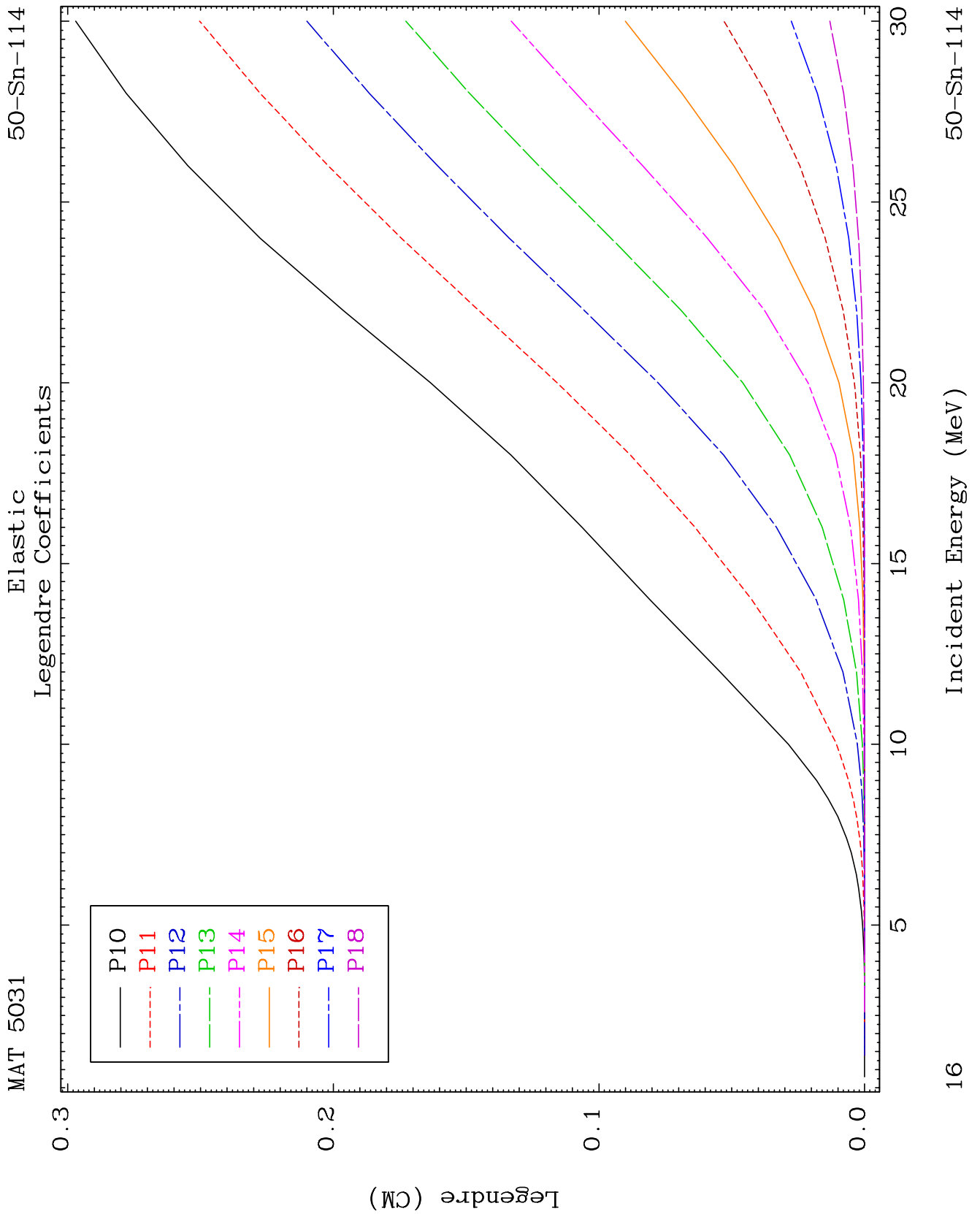
MAT 5031

(n, α) Levels
293 Kelvin Cross Sections

50-Sn-114



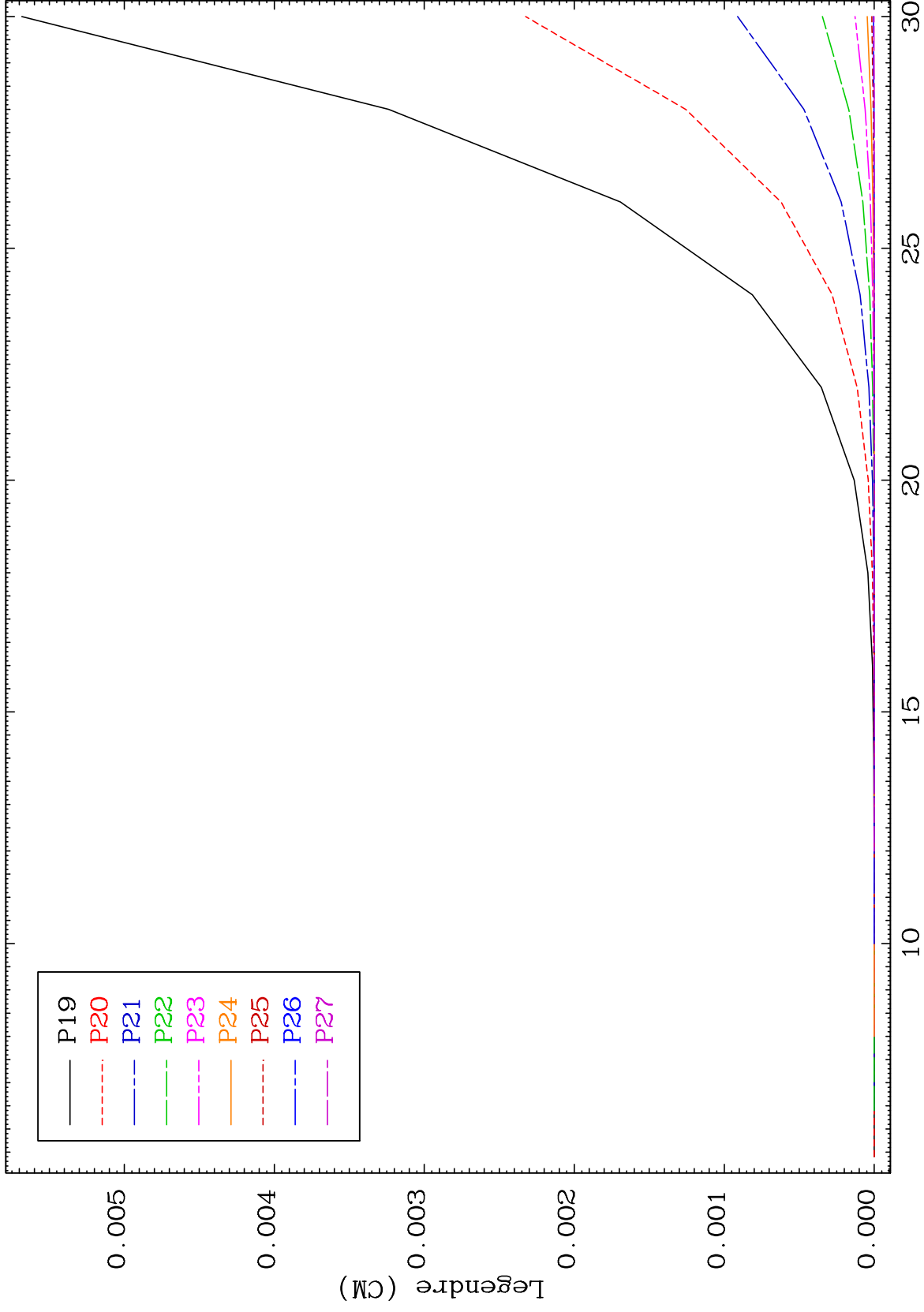




MAT 5031

Elastic
Legendre Coefficients

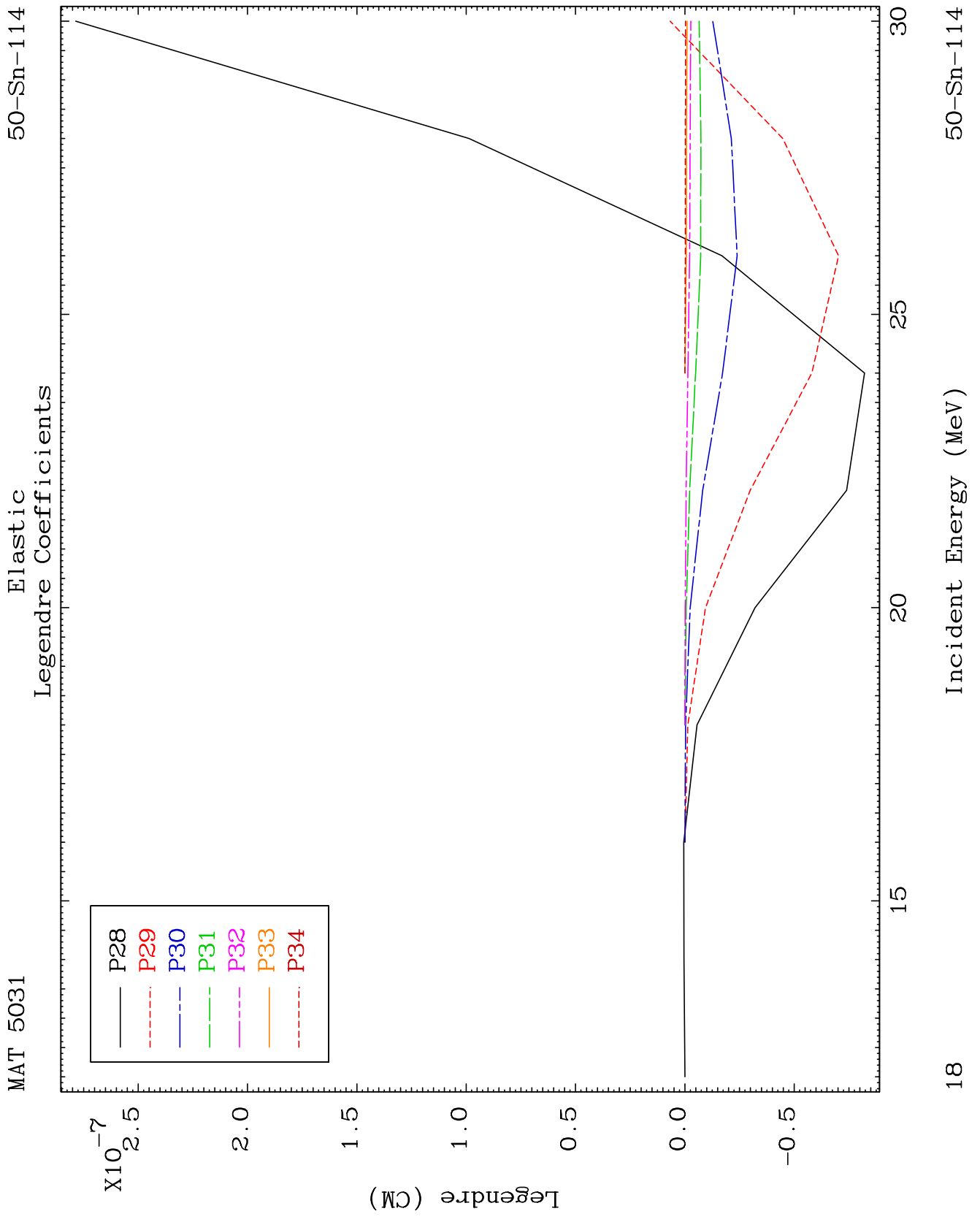
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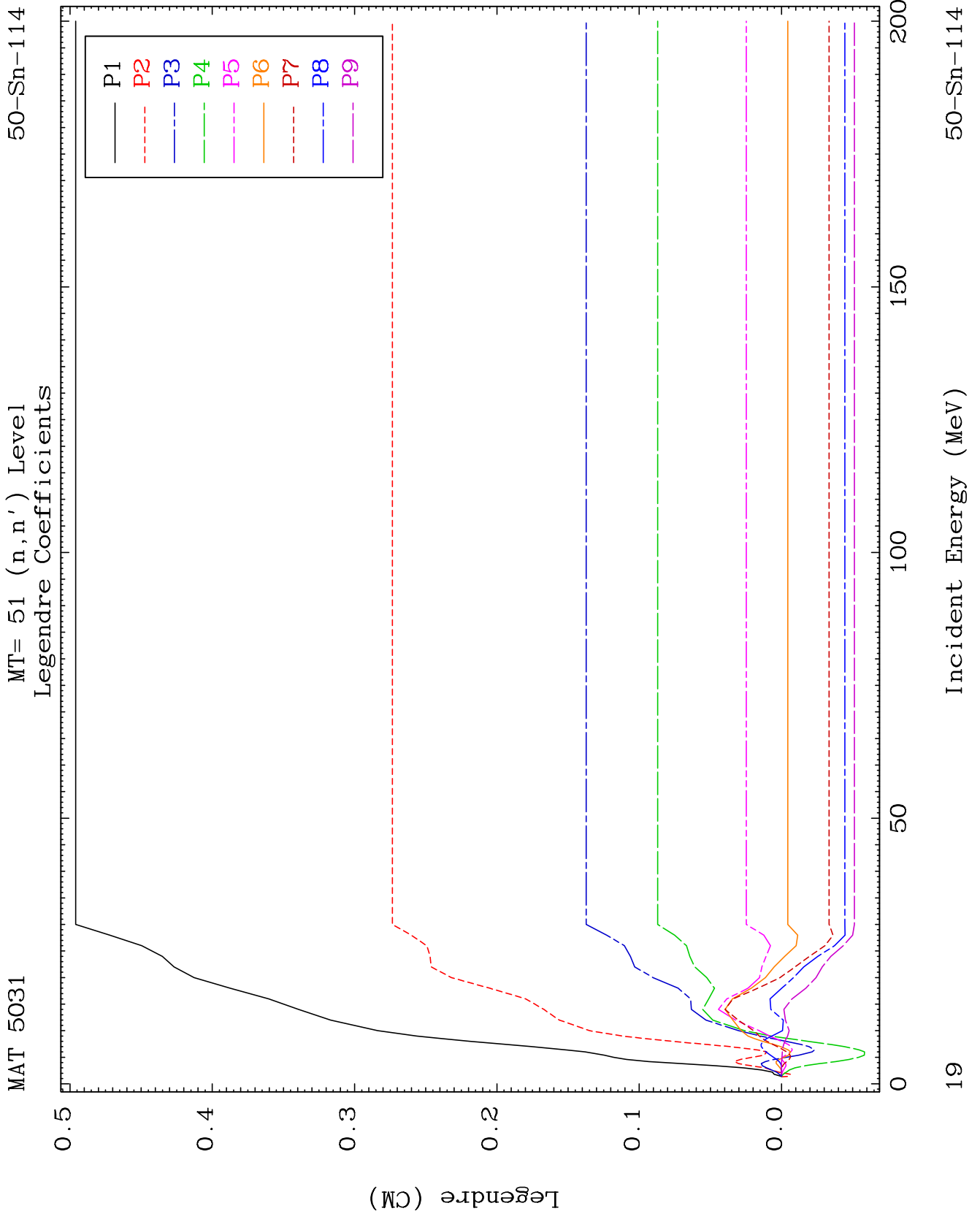


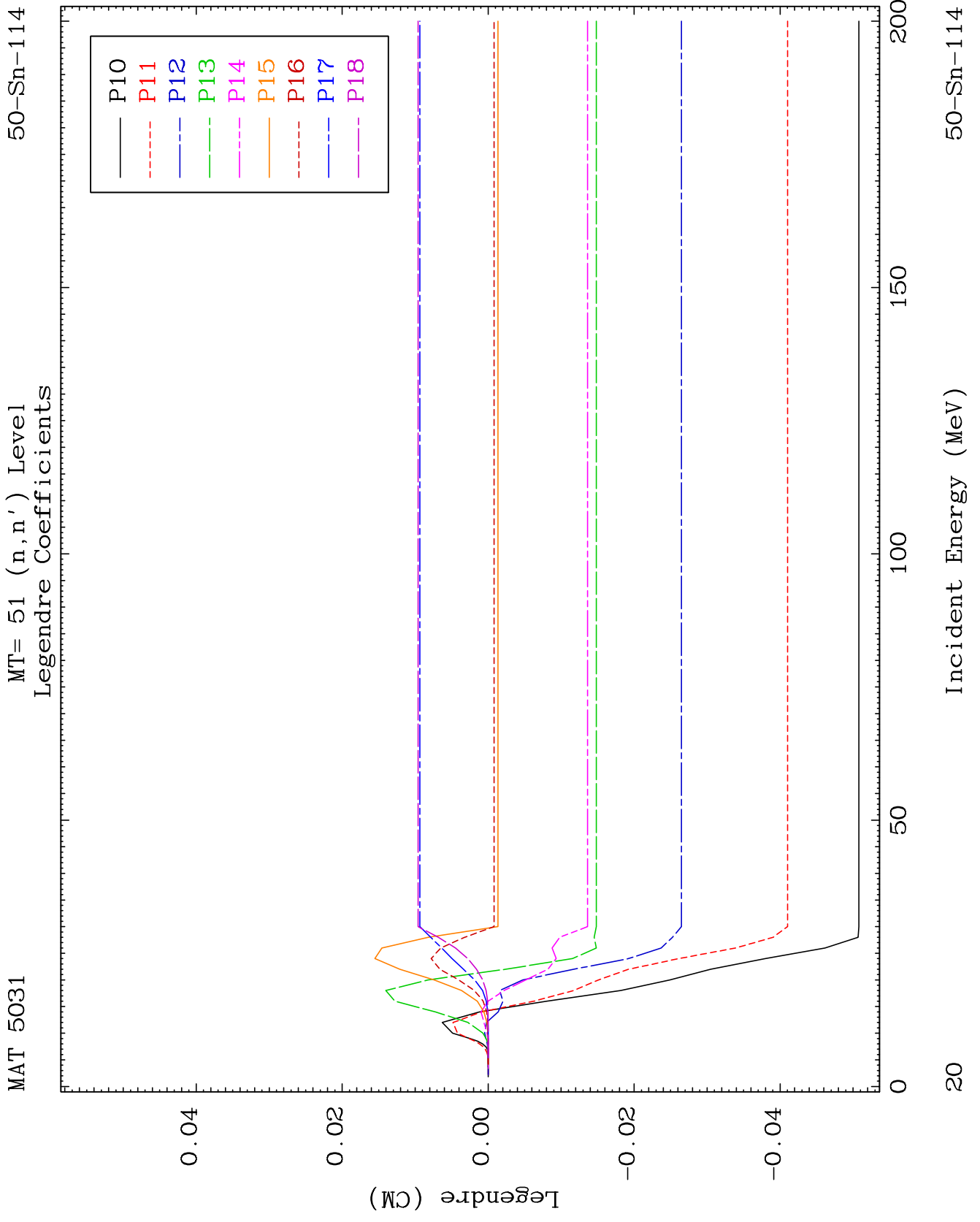
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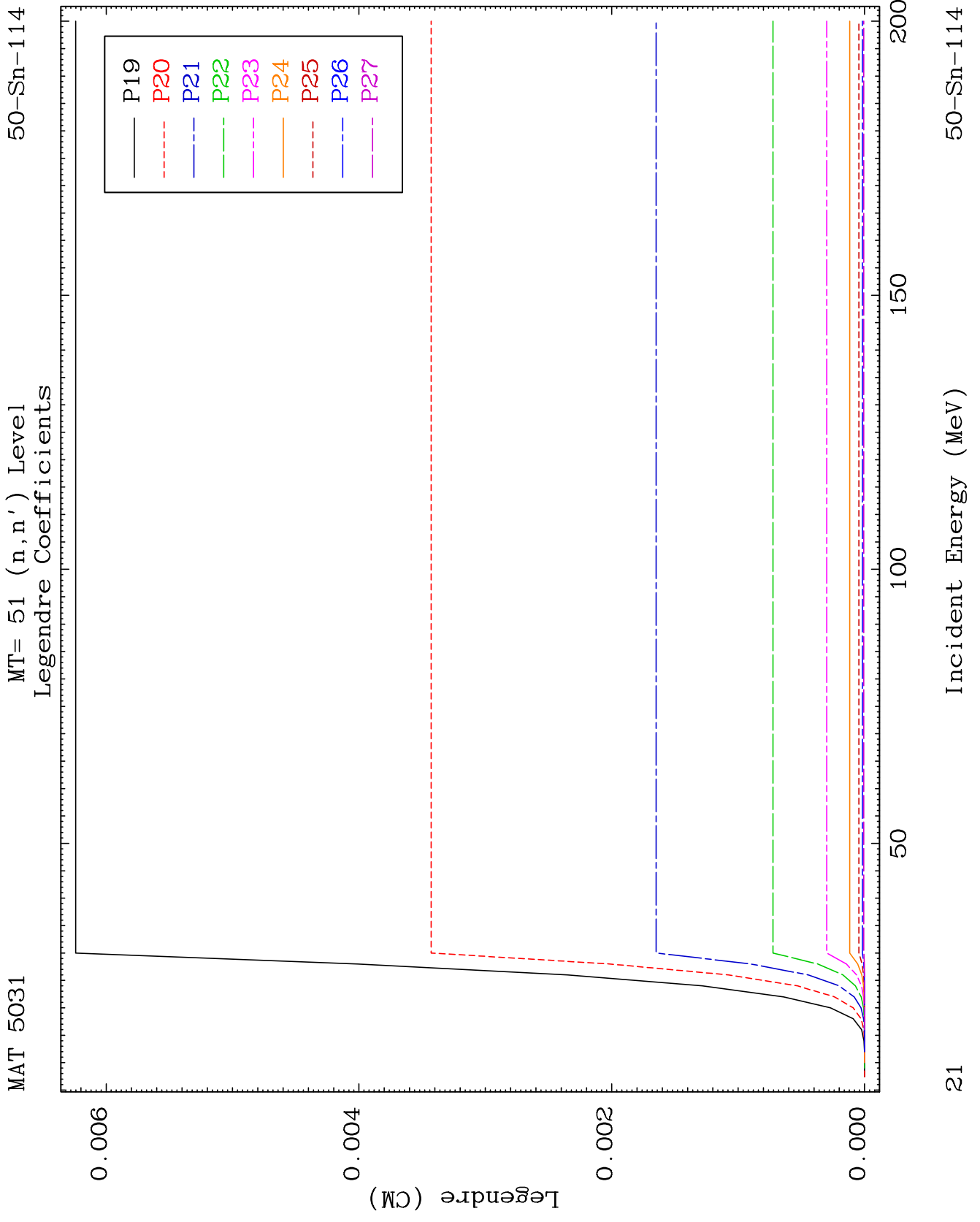
Incident Energy (MeV)

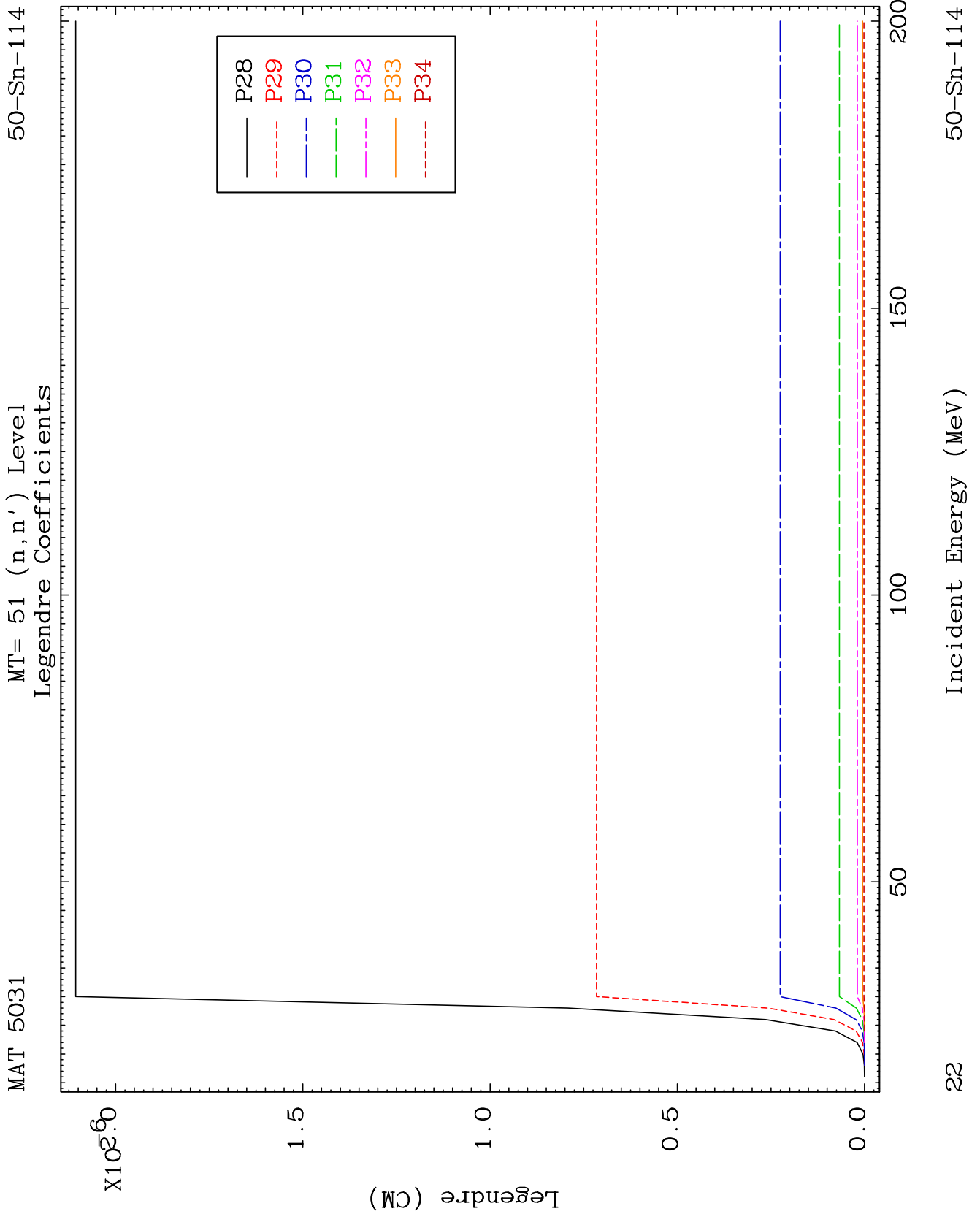
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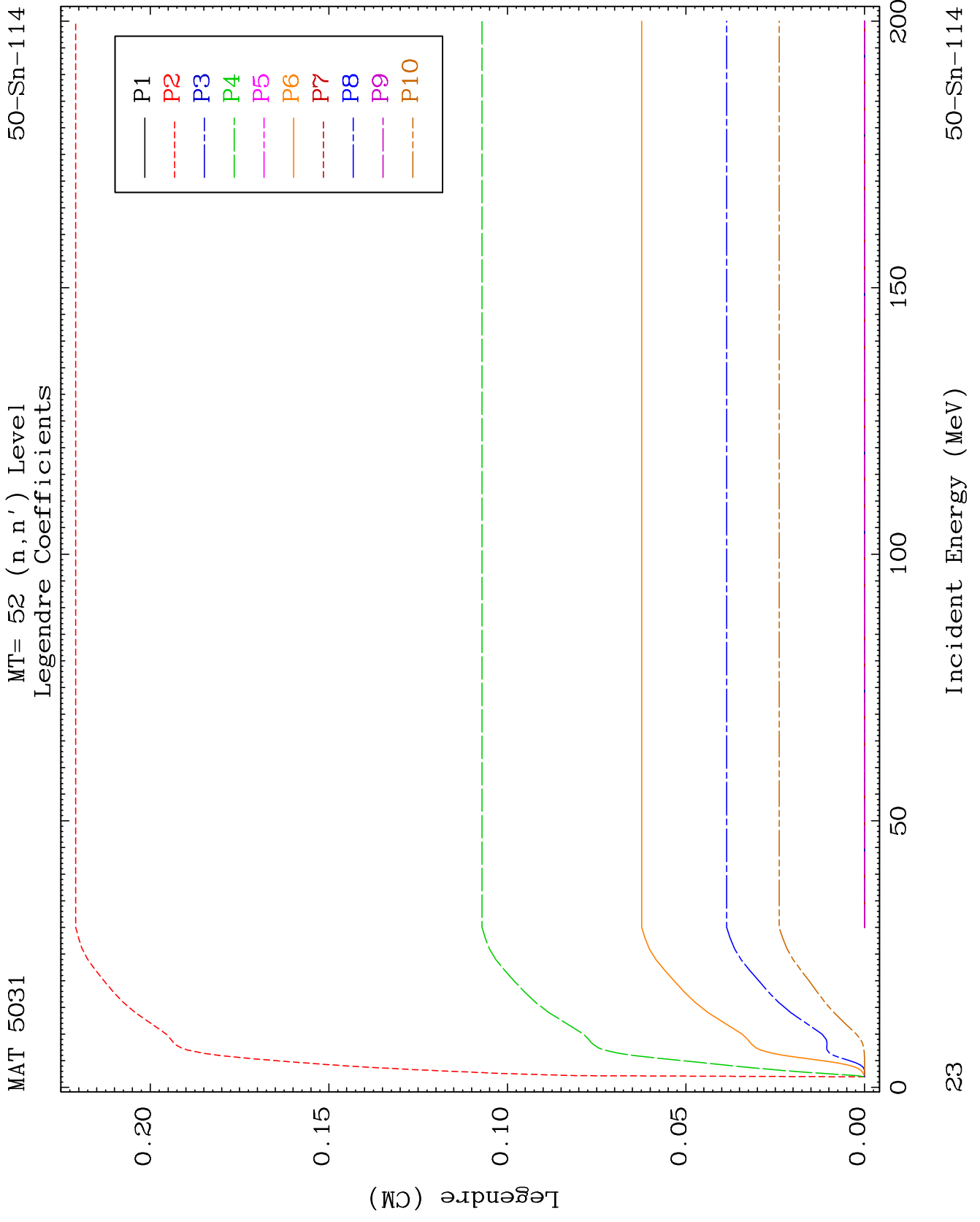


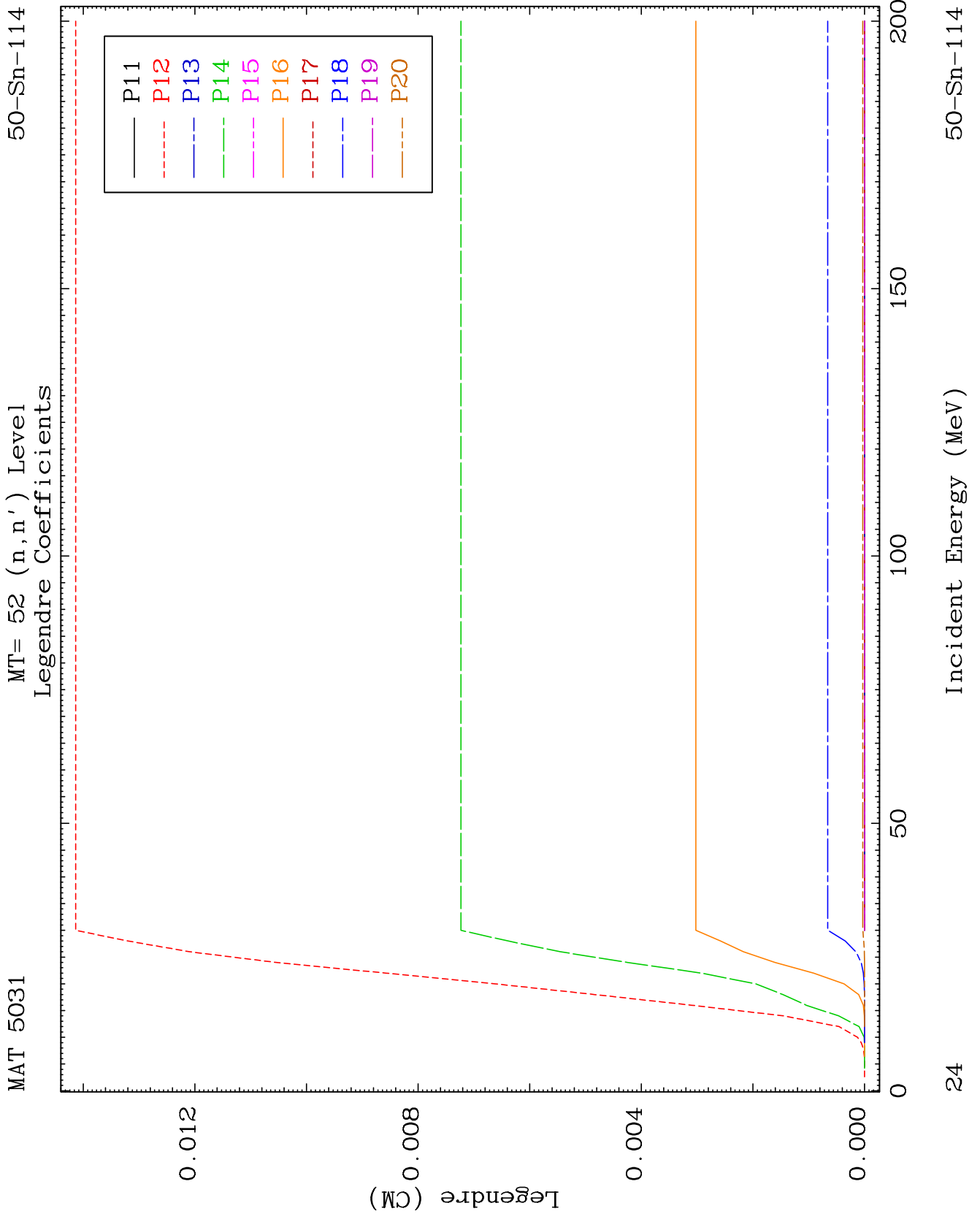


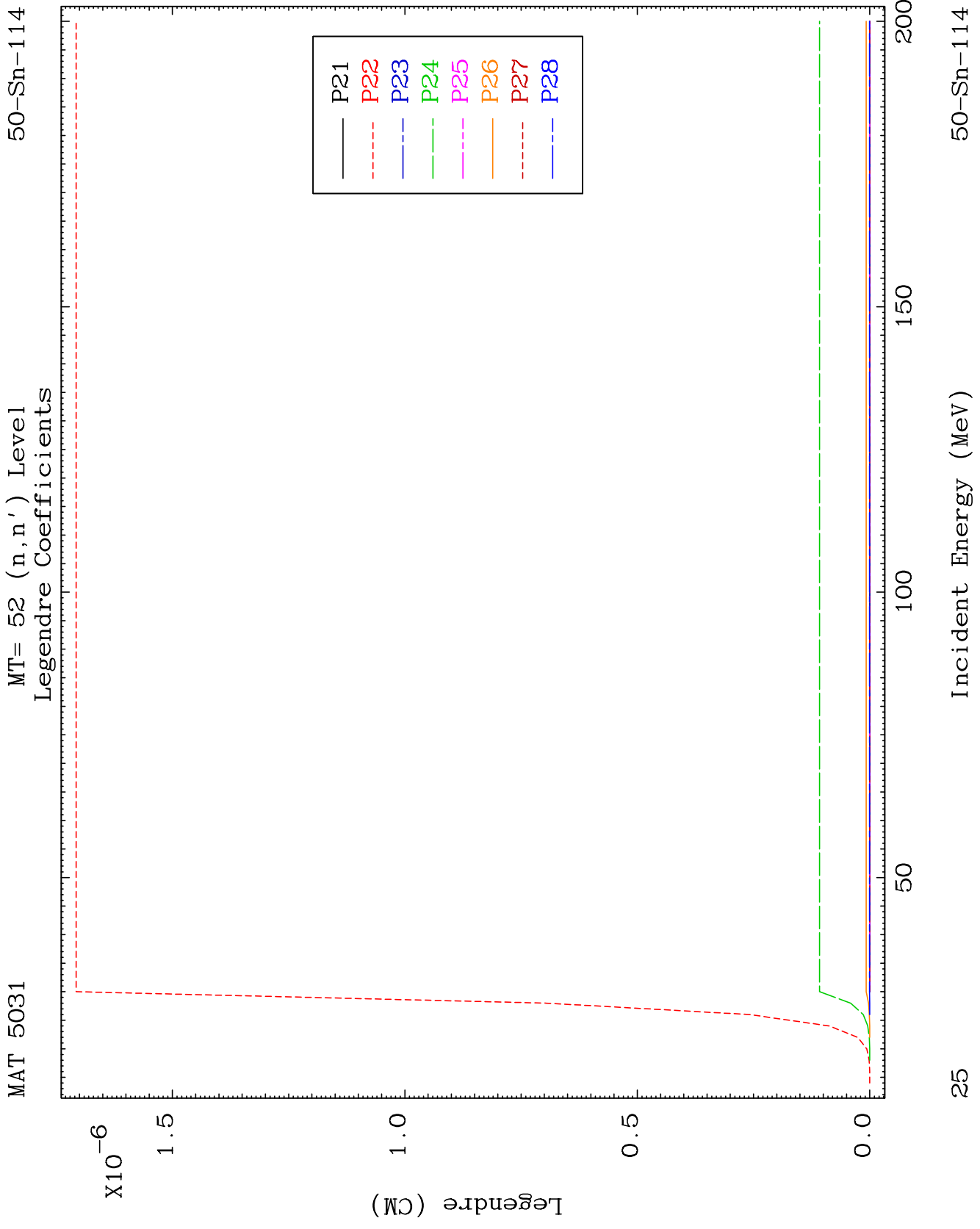


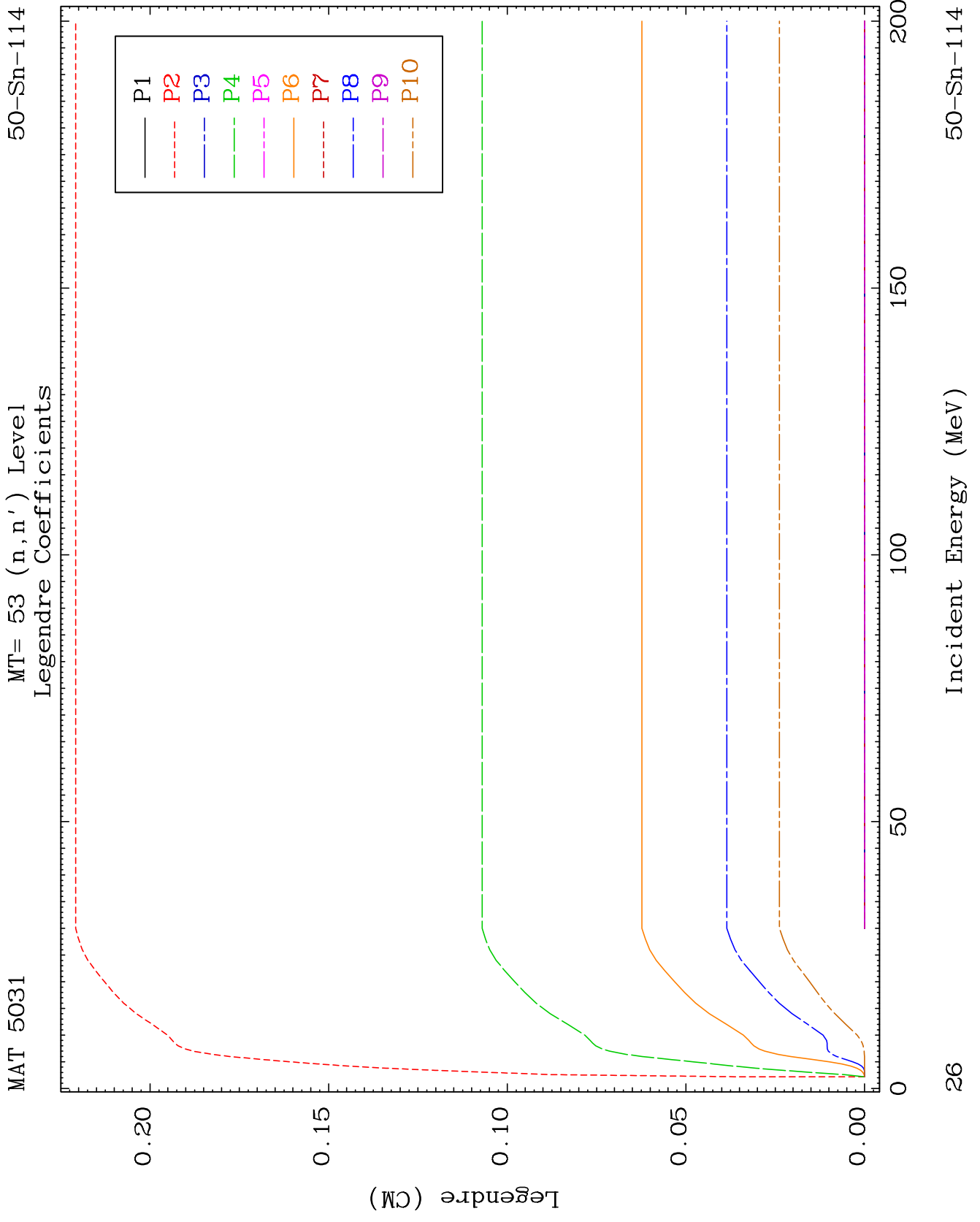


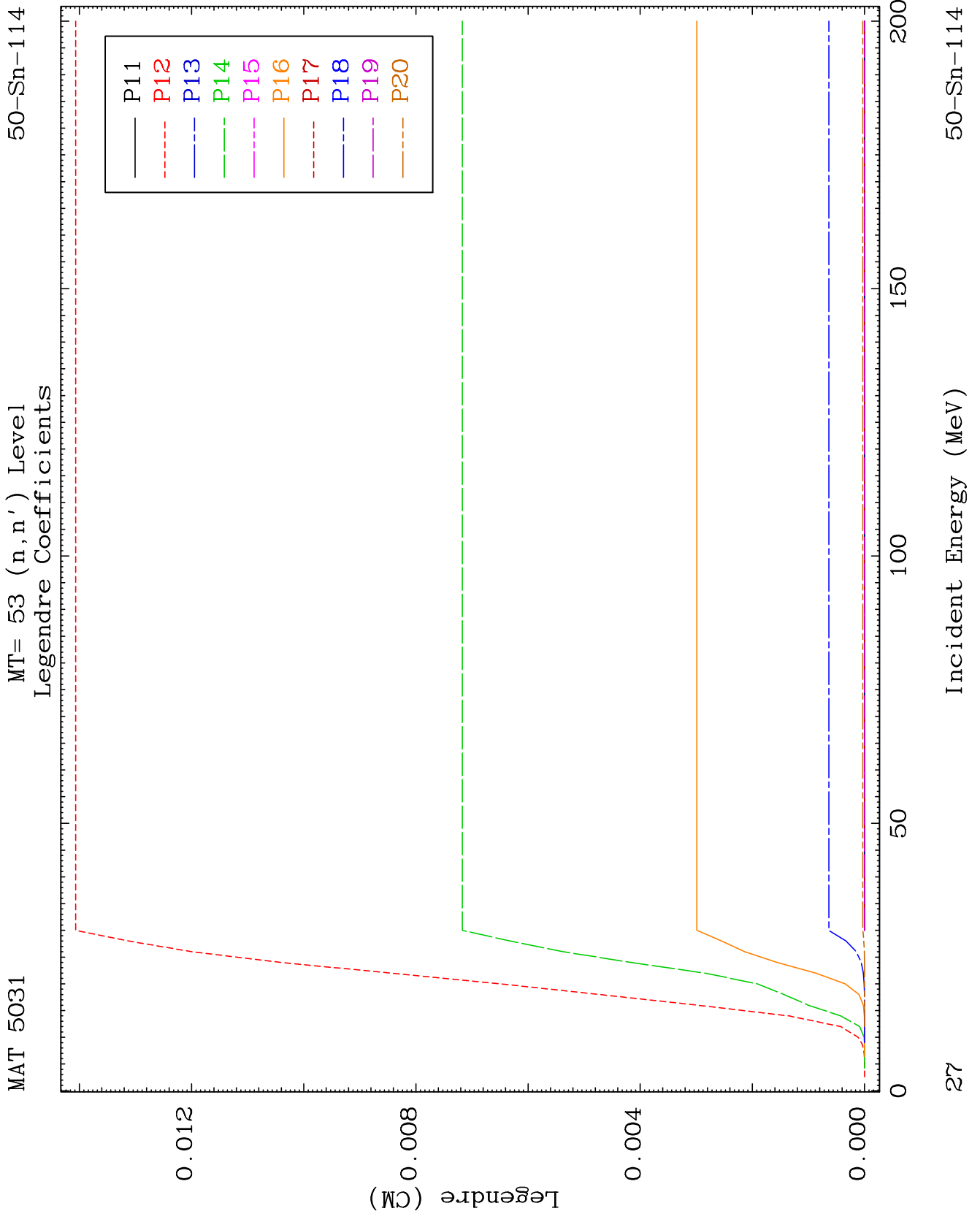


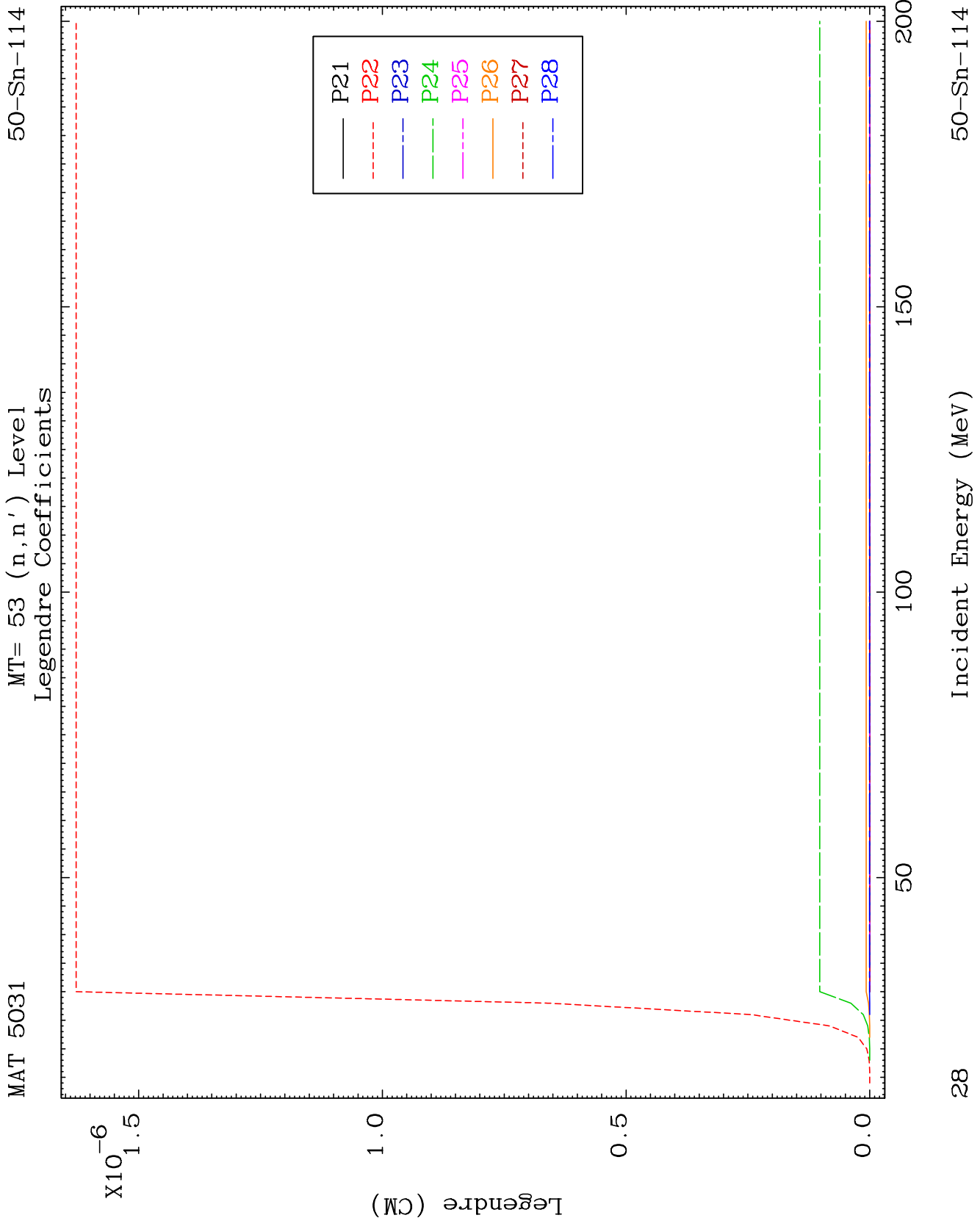








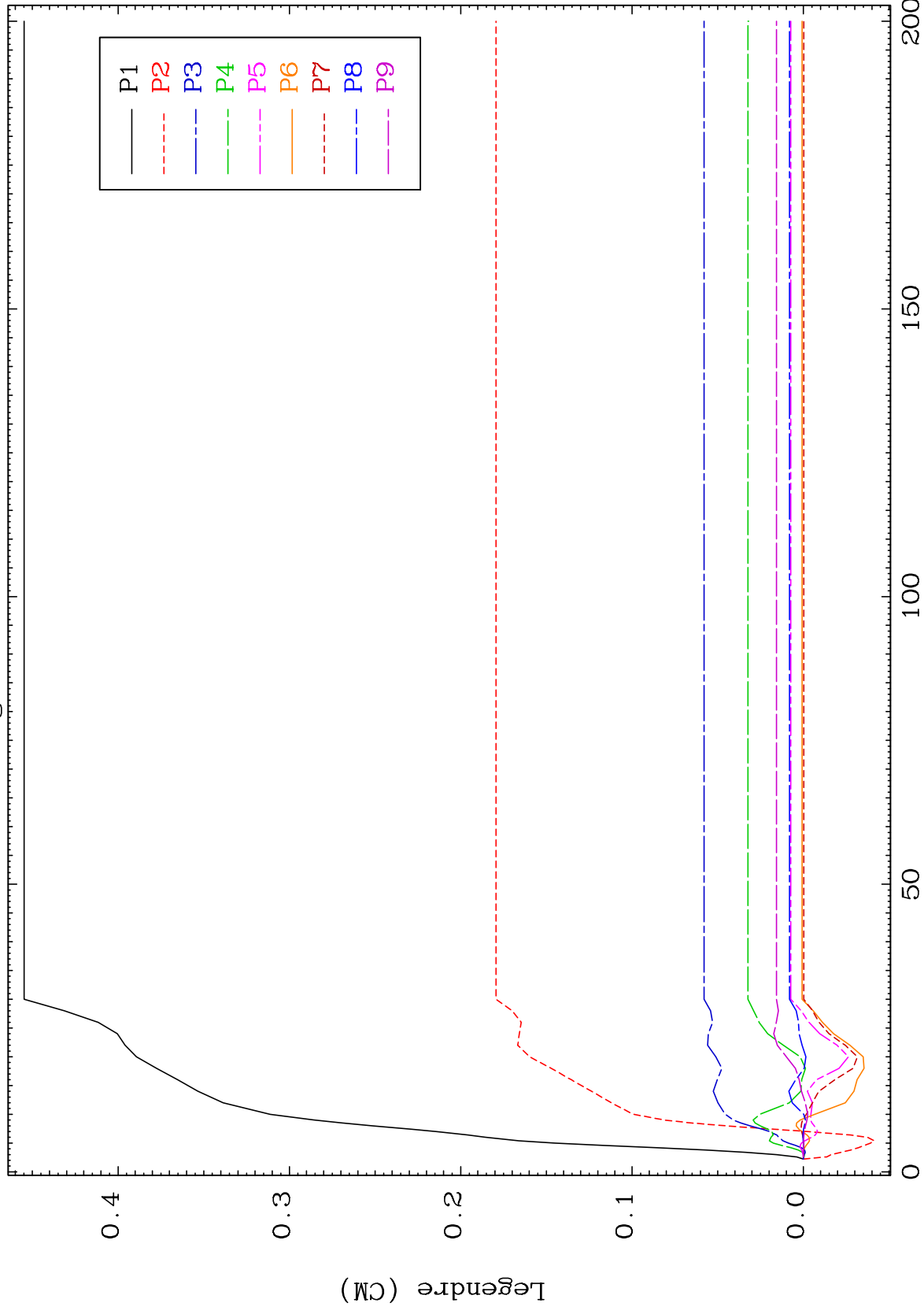




MAT 5031

MT= 54 (n,n') Level
Legendre Coefficients

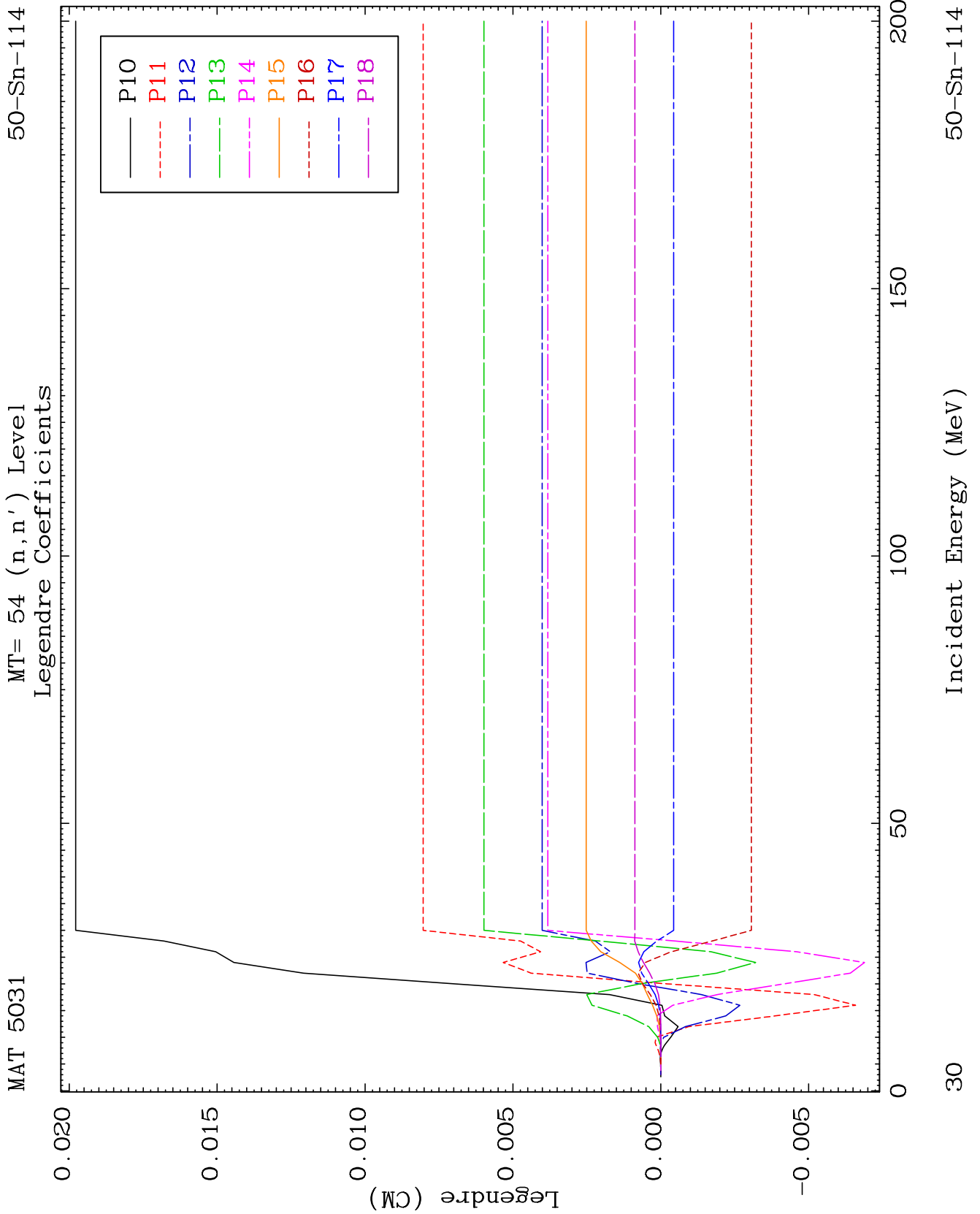
50-Sn-114



29

Incident Energy (MeV)

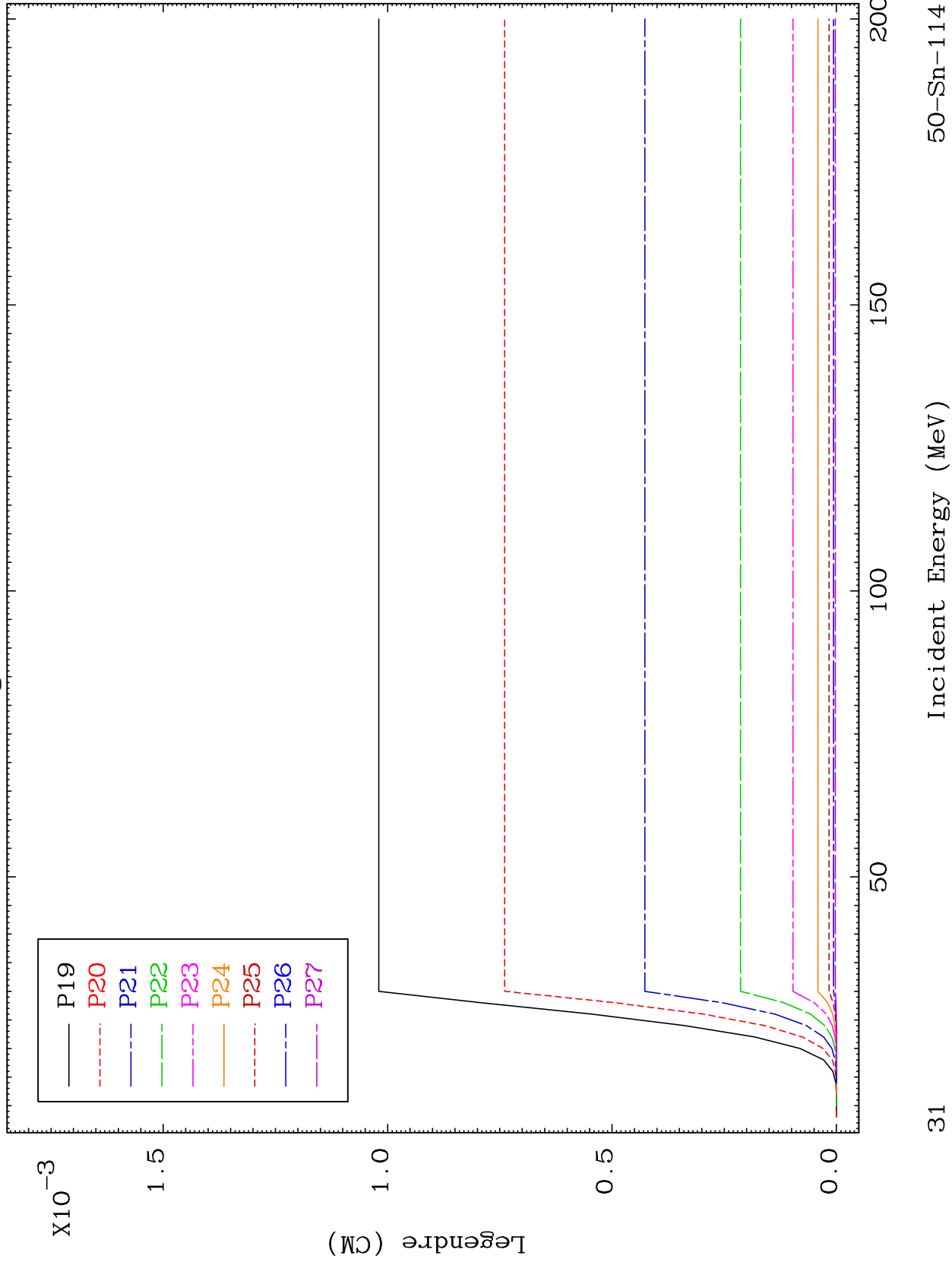
50-Sn-114



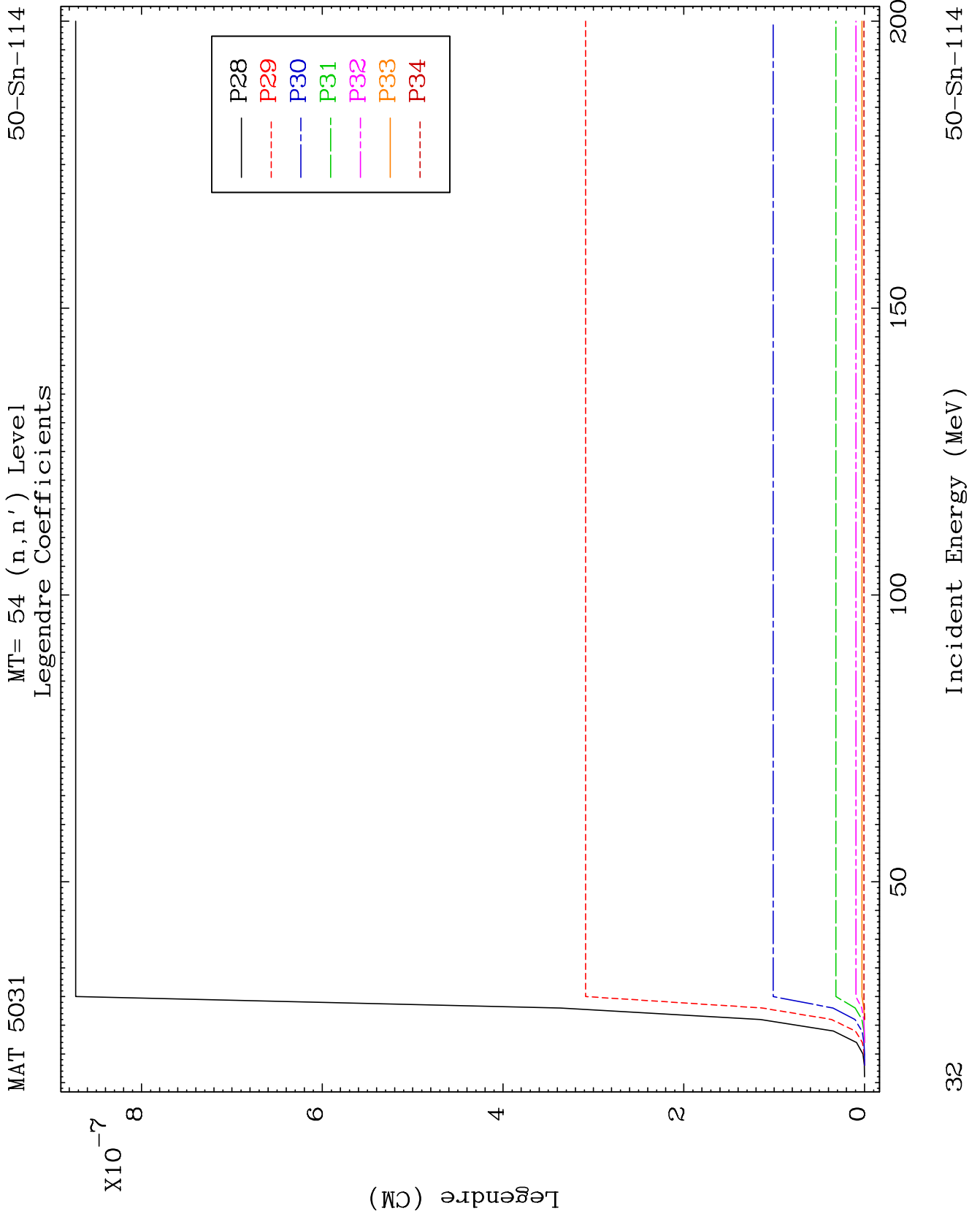
MAT 5031

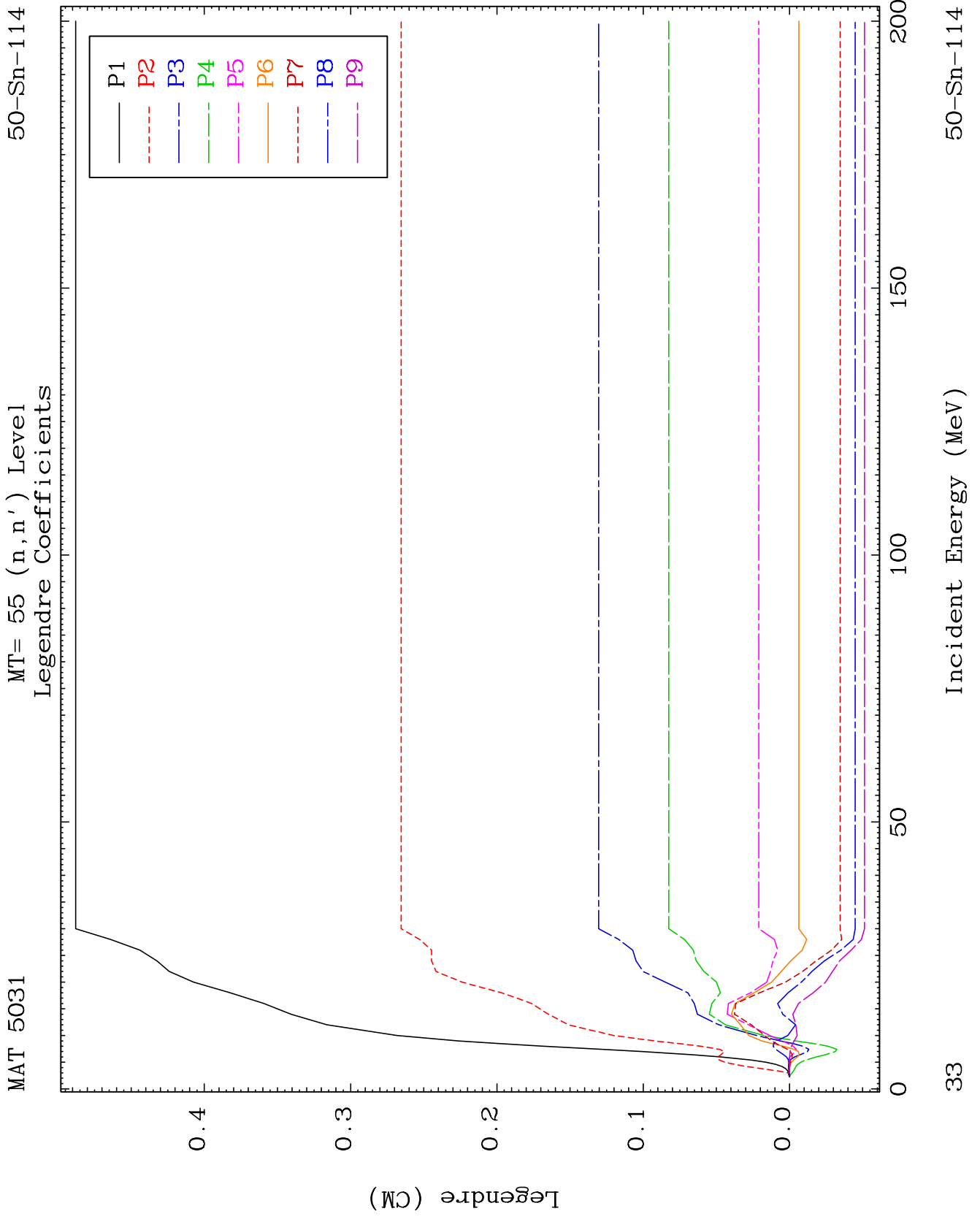
MT= 54 (n,n') Level
Legendre Coefficients

50-Sn-114



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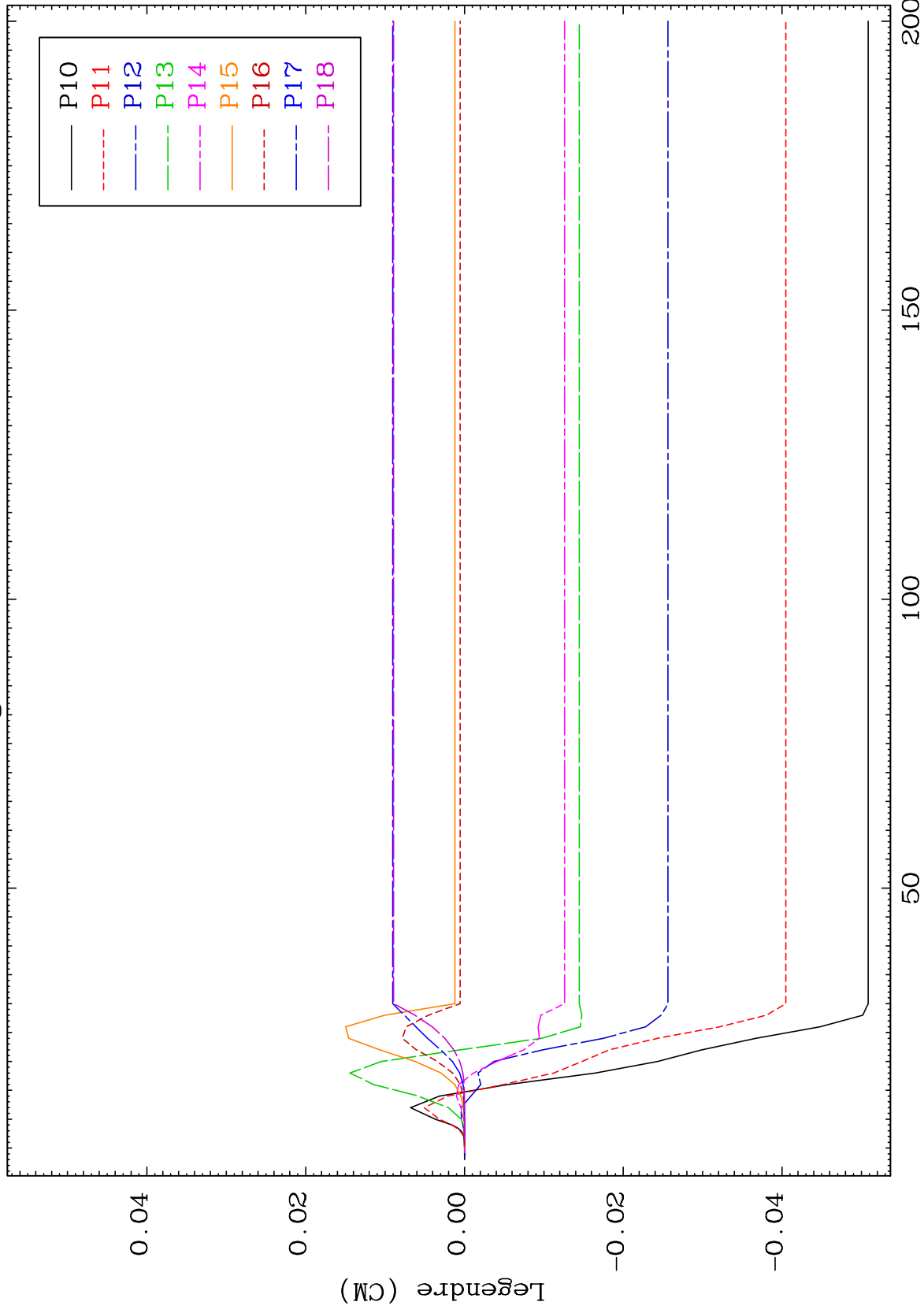




MAT 5031

MT= 55 (n,n') Level
Legendre Coefficients

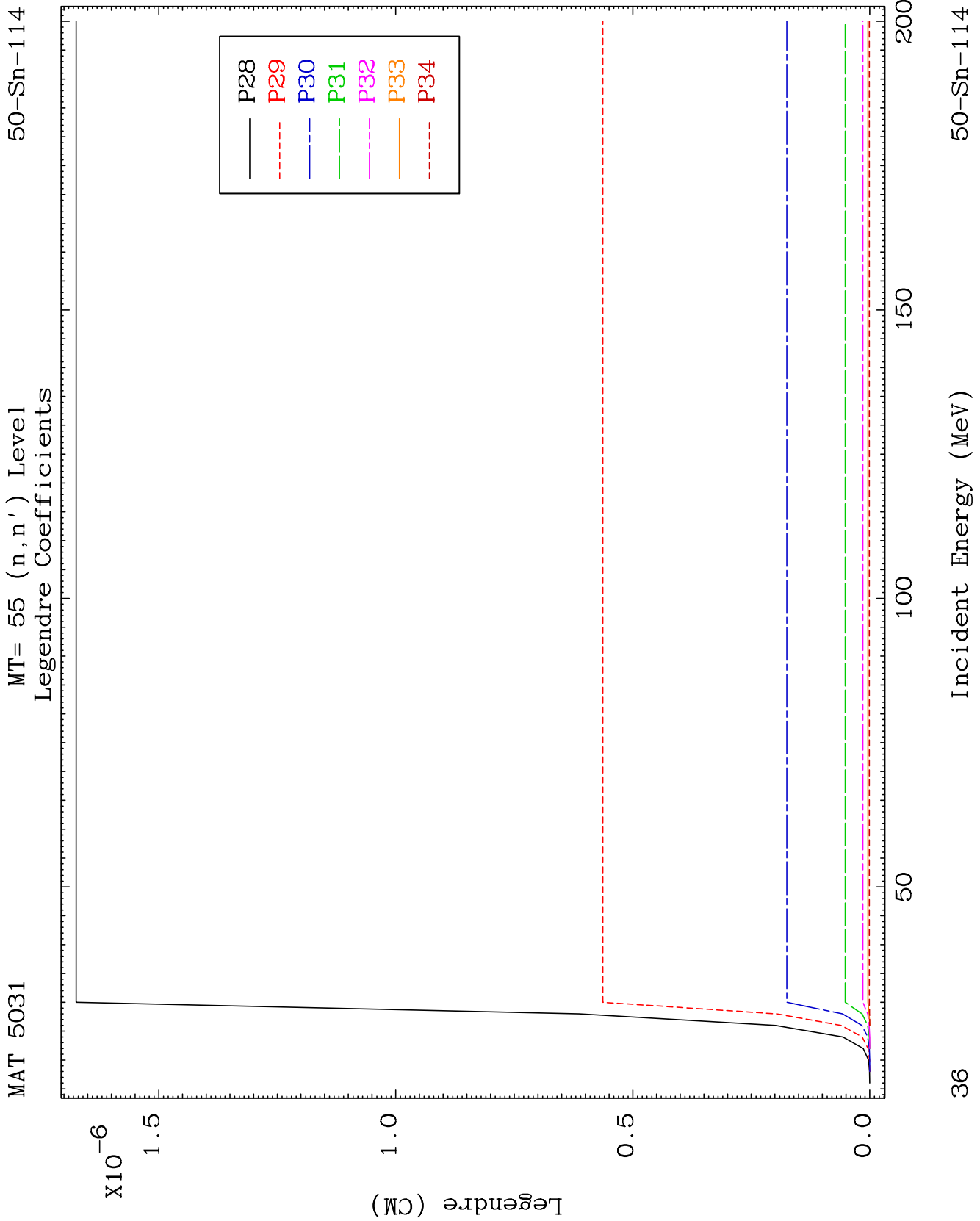
50-Sn-114



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Incident Energy (MeV)

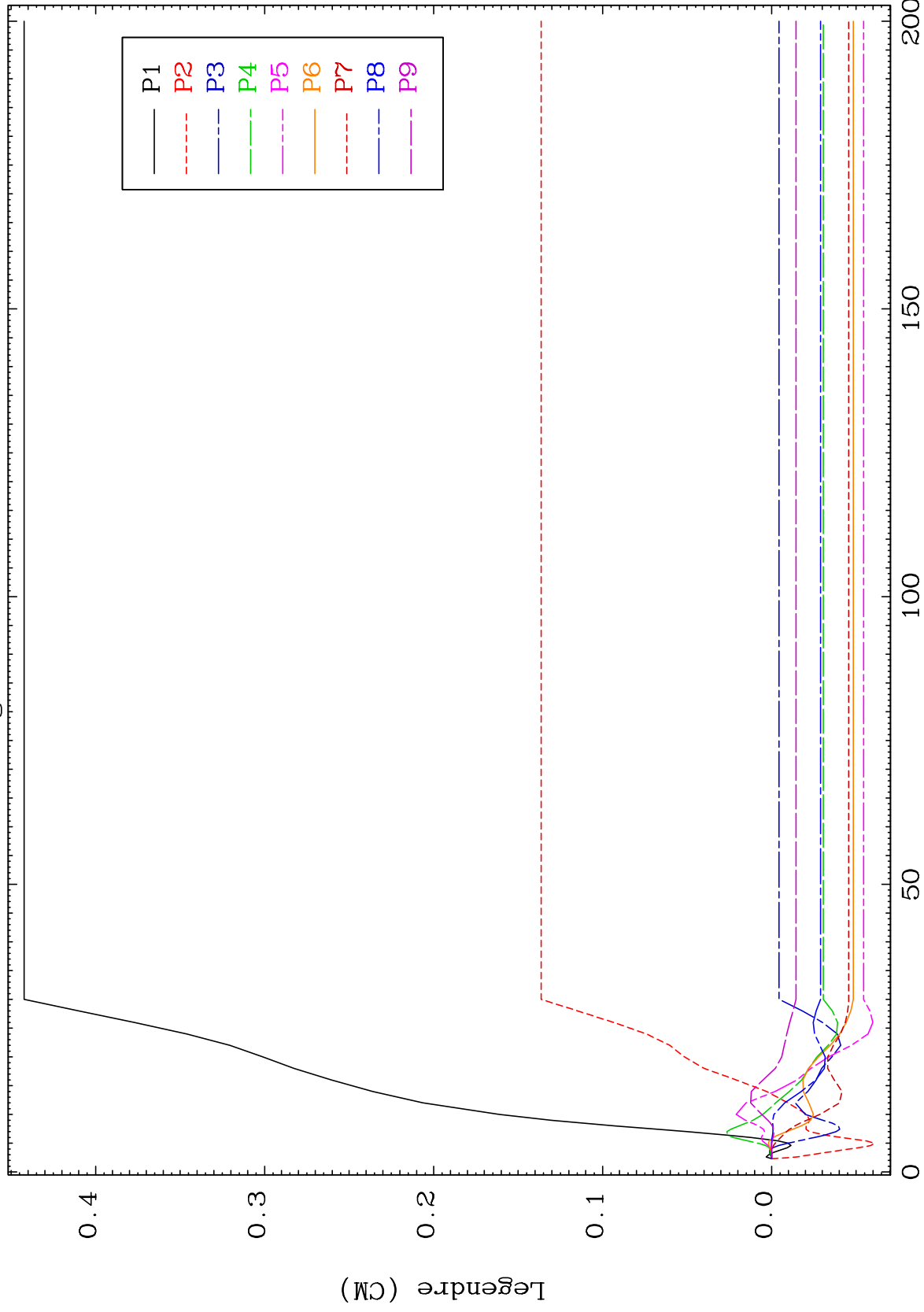
50-Sn-114



MAT 5031

MT= 56 (n,n') Level
Legendre Coefficients

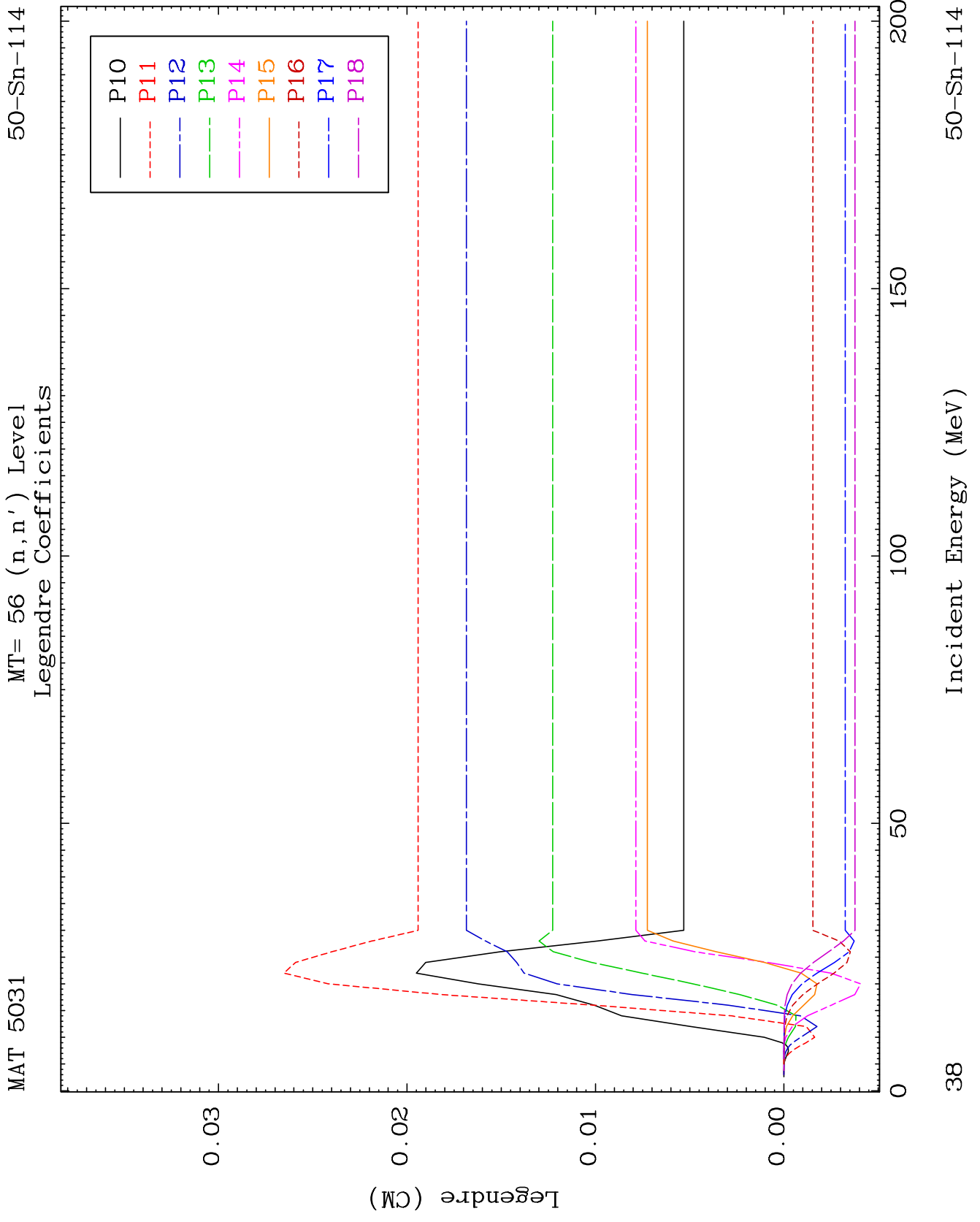
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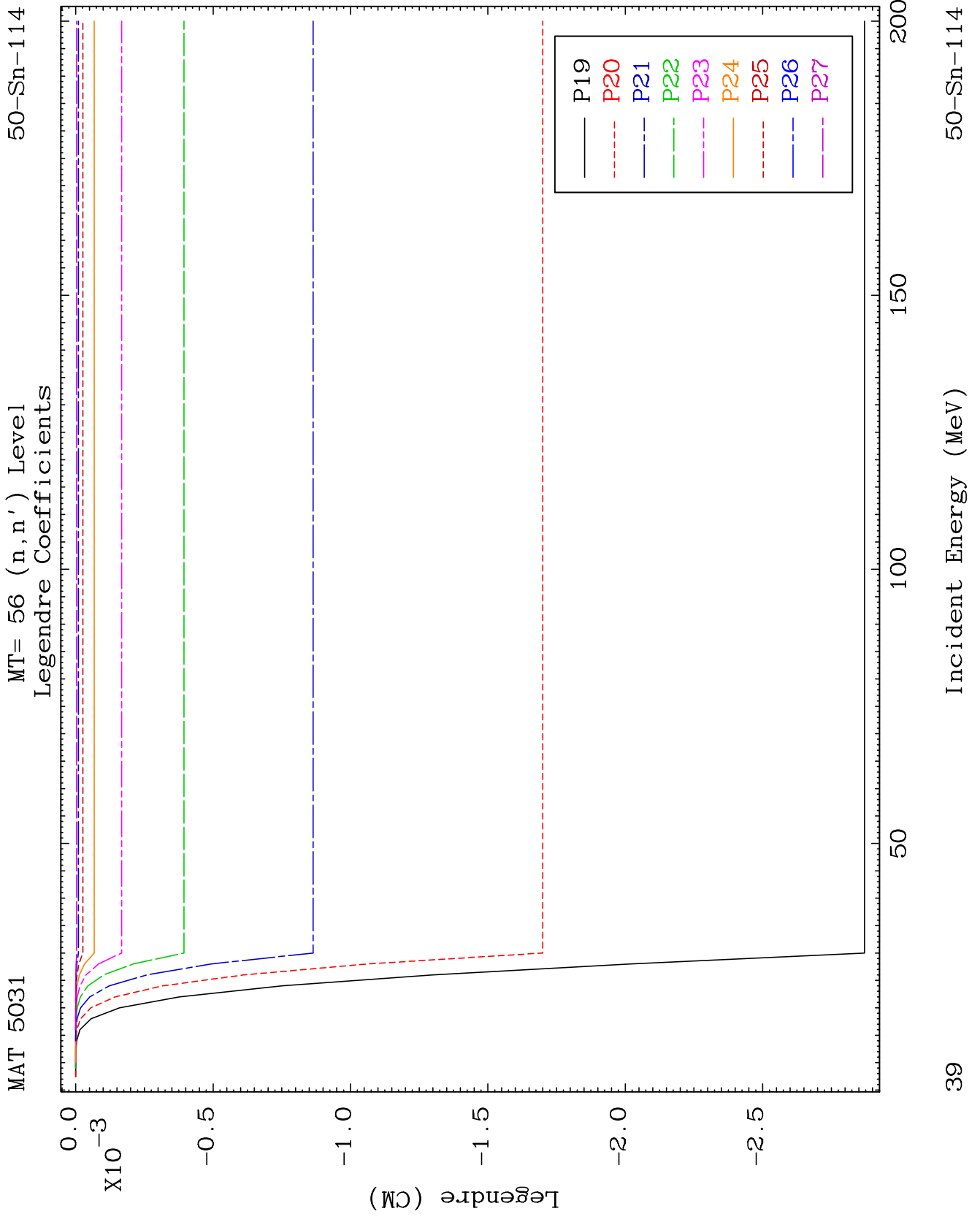


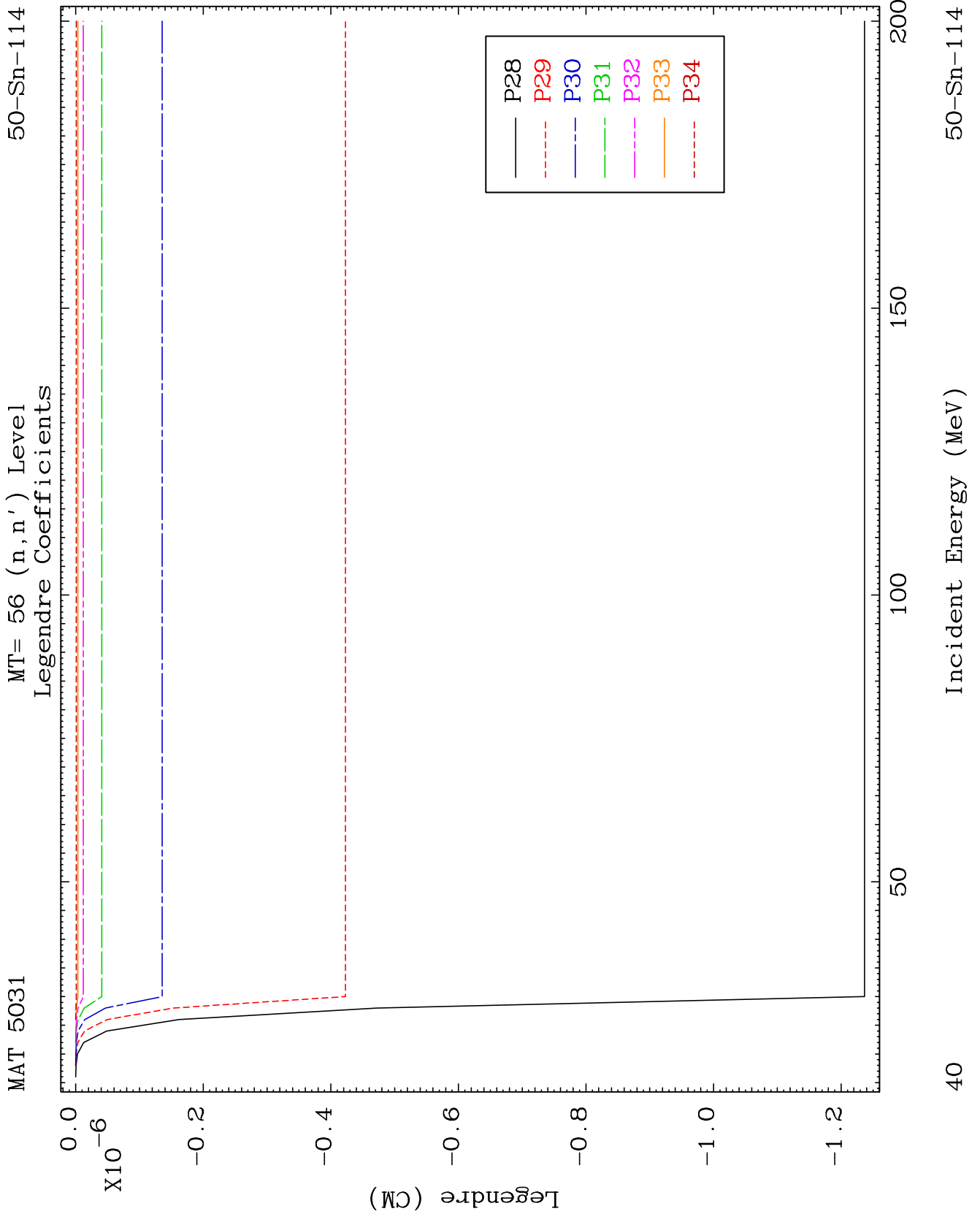
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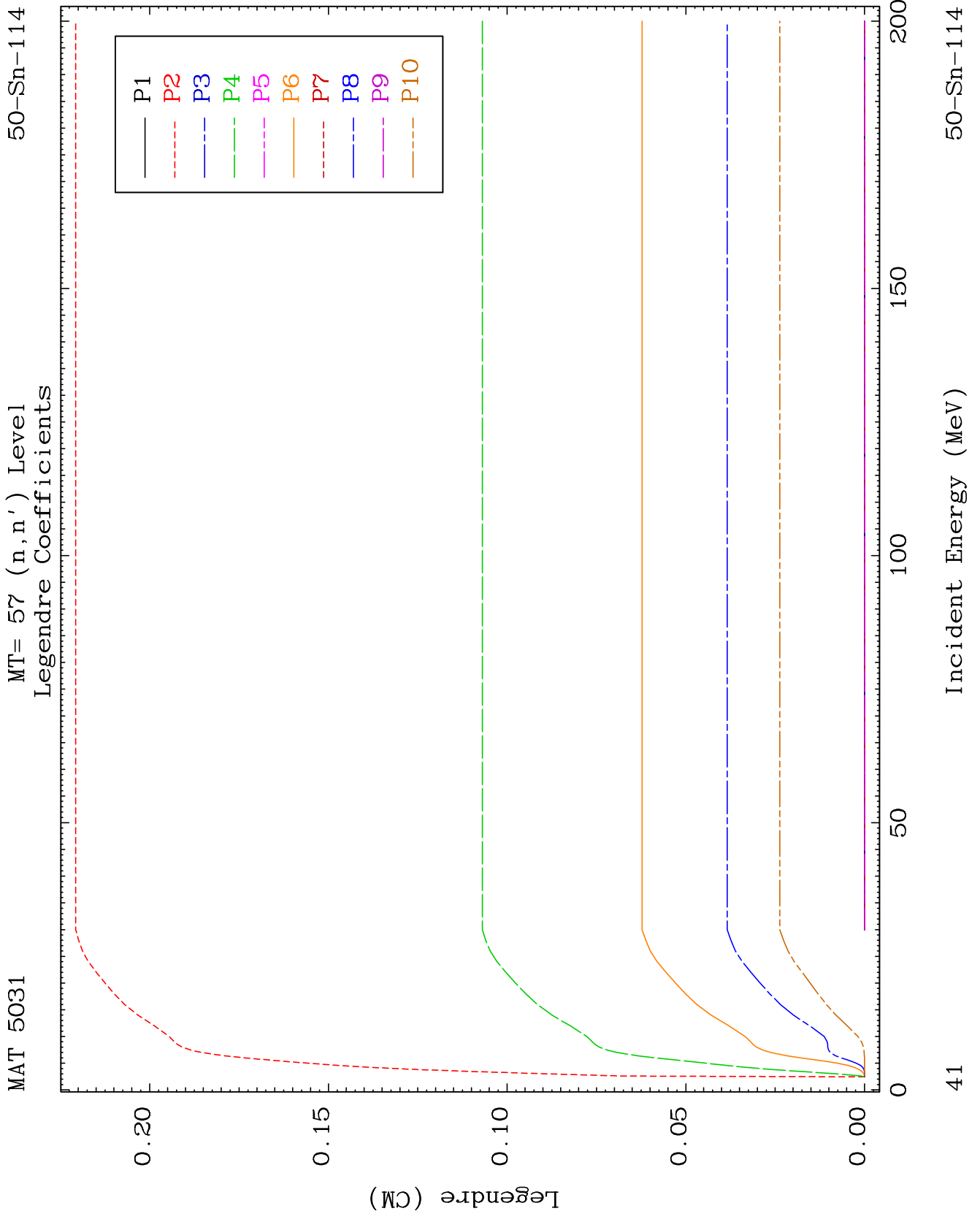
Incident Energy (MeV)

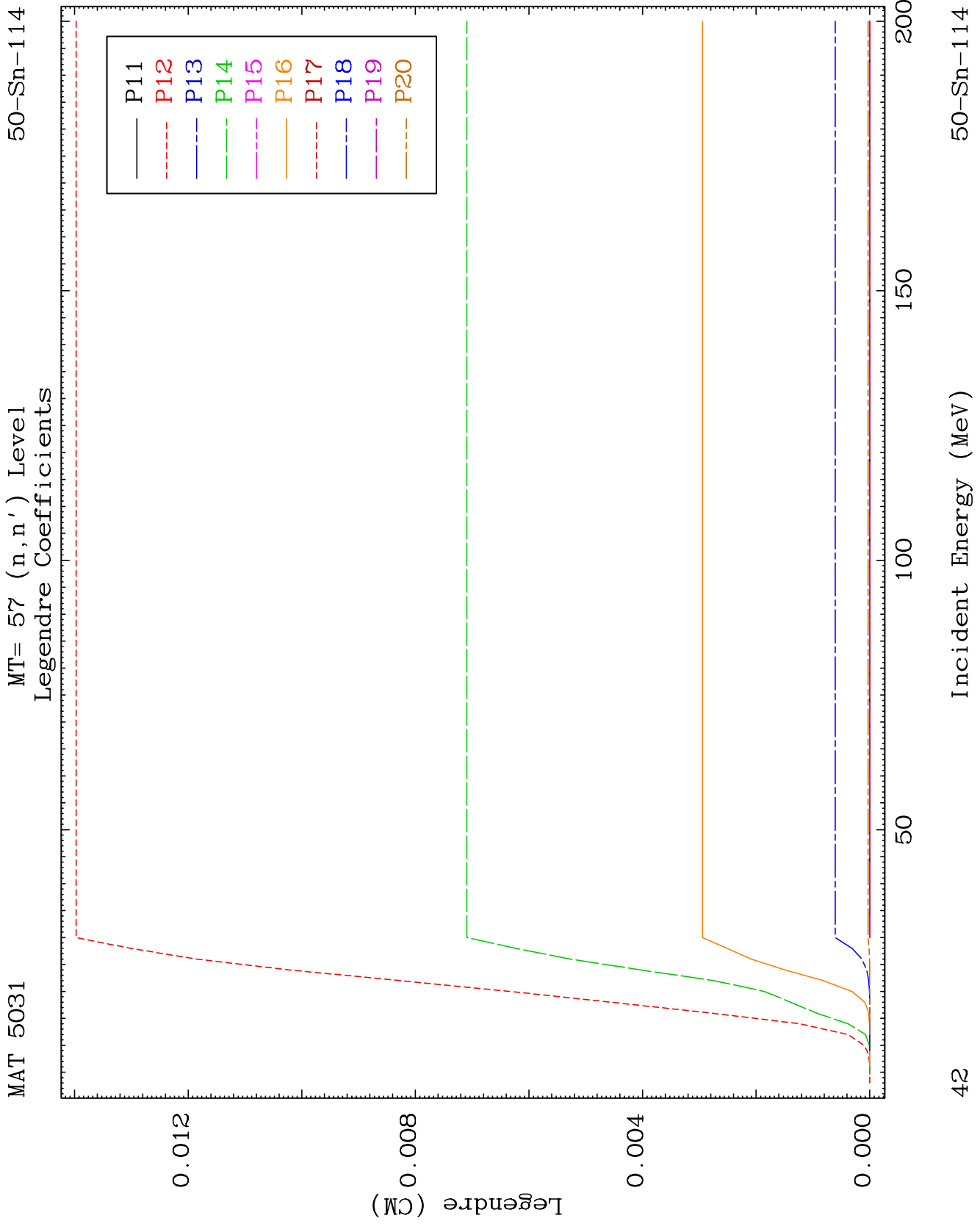
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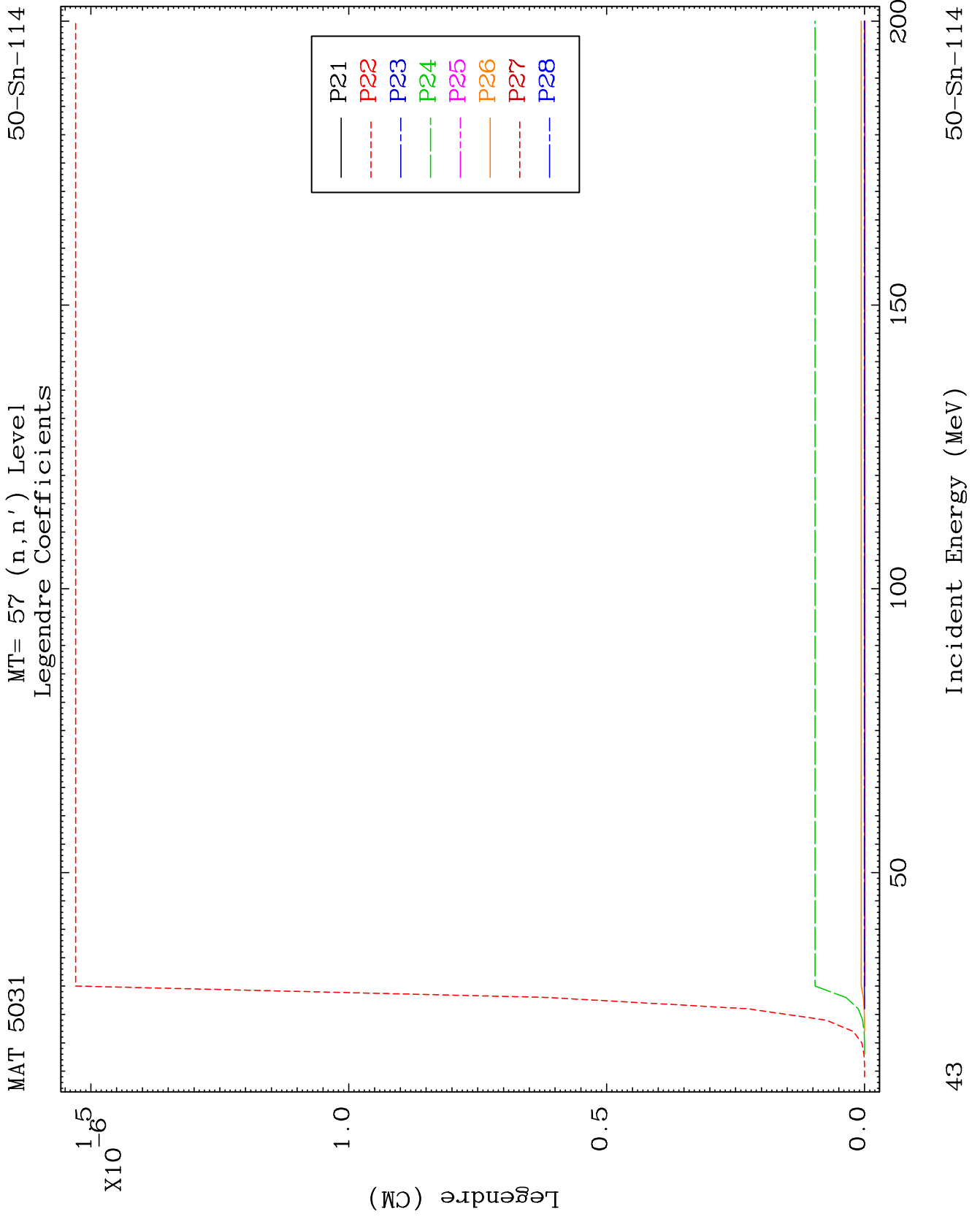


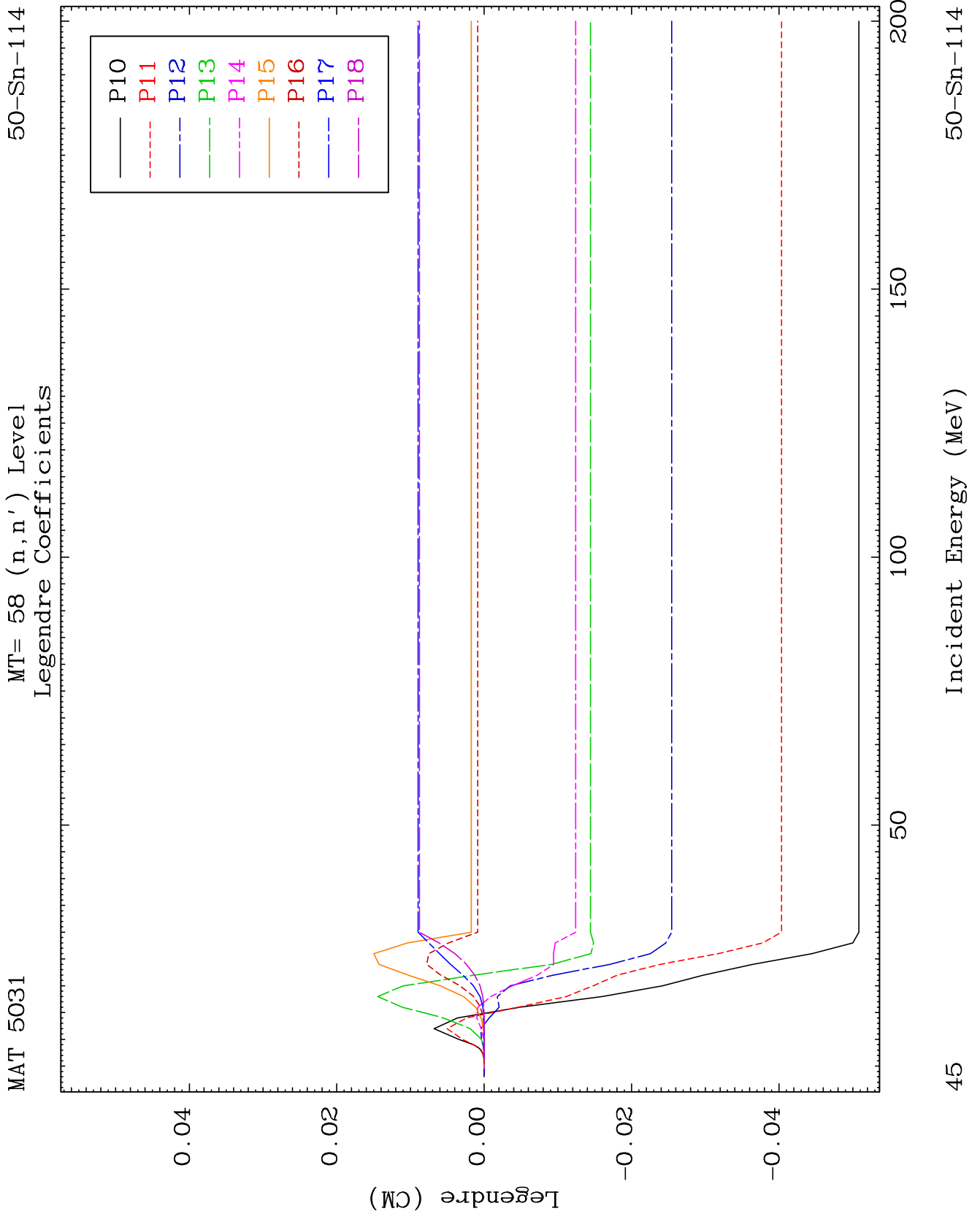








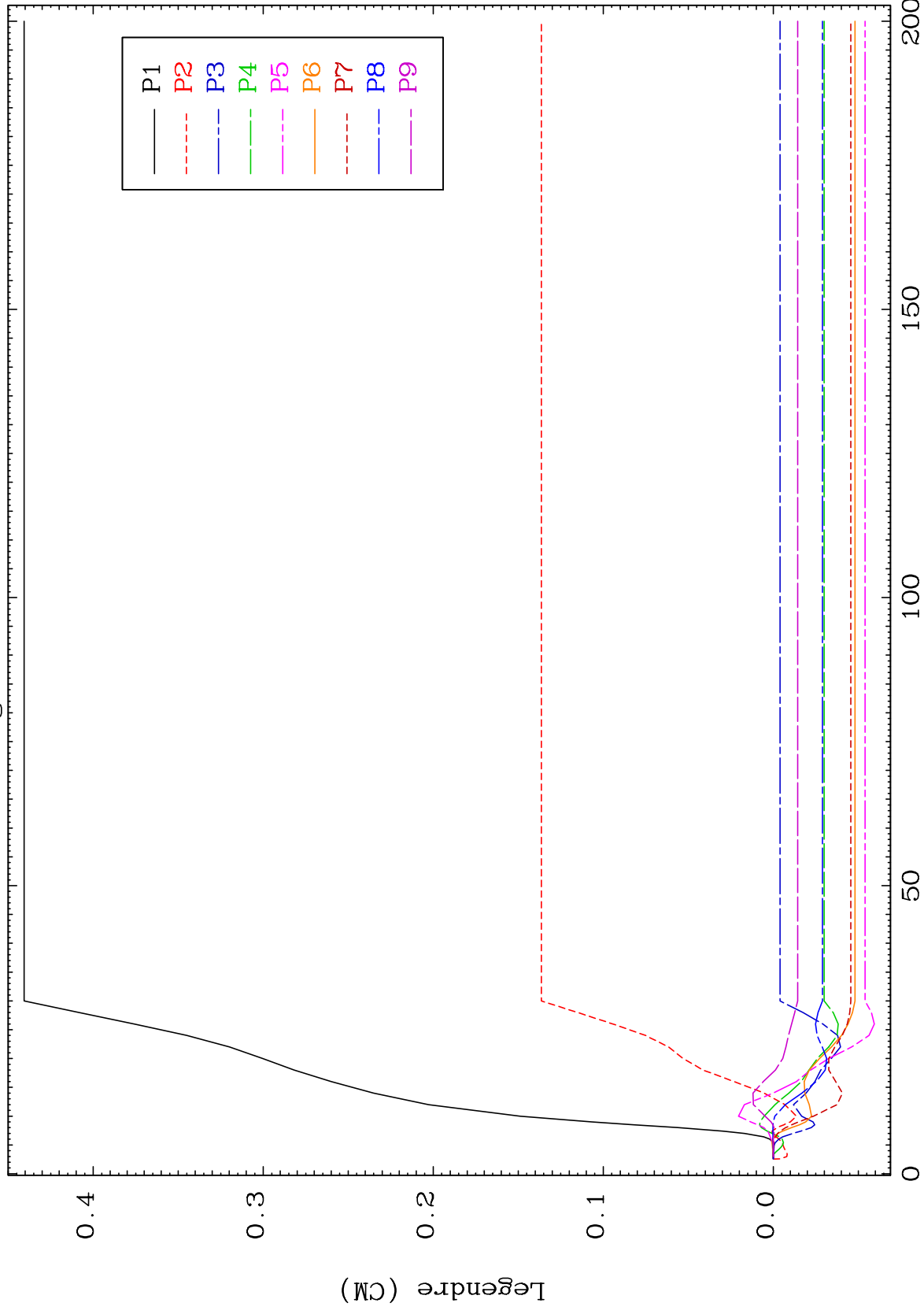




MAT 5031

MT= 59 (n,n') Level
Legendre Coefficients

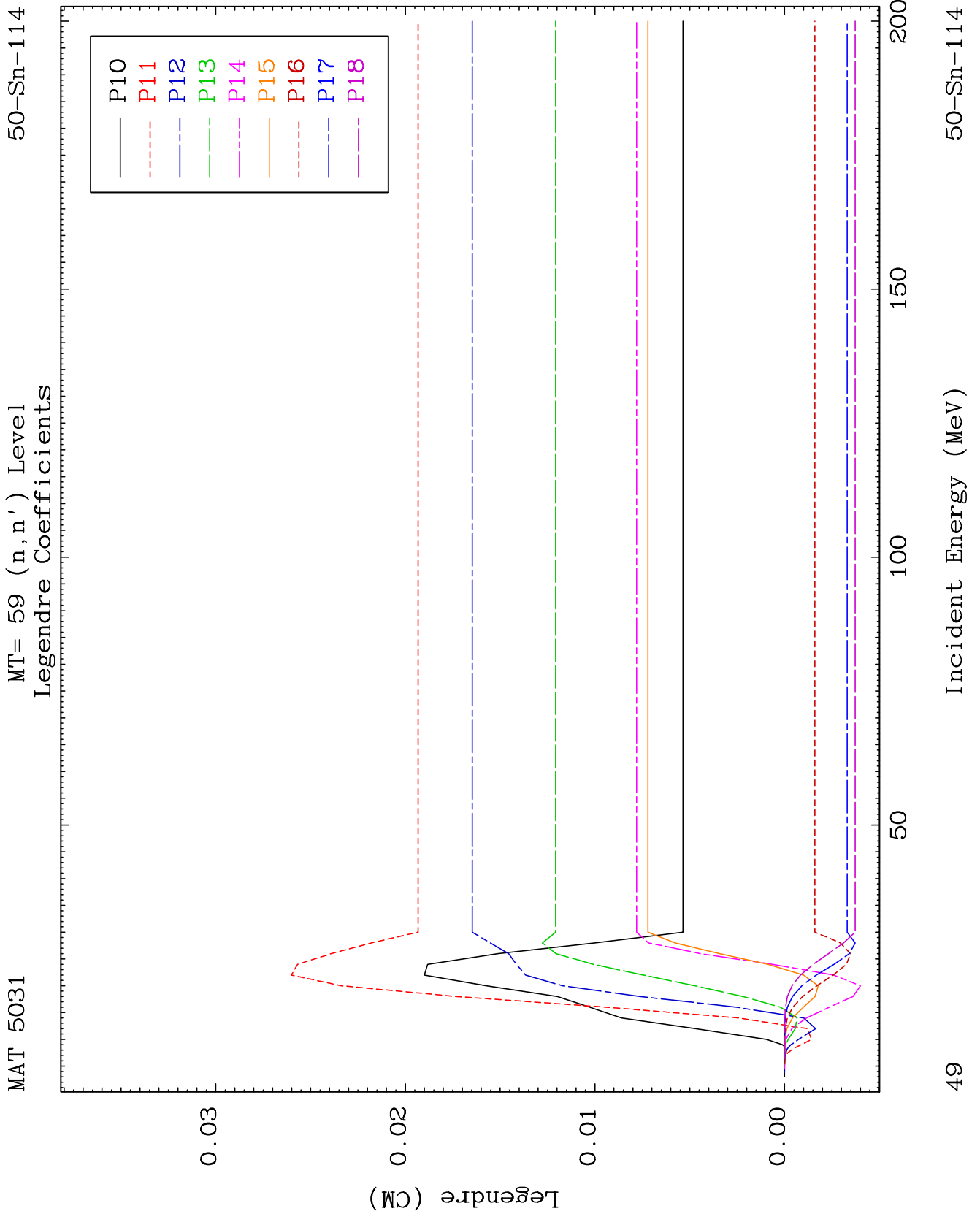
50-Sn-114

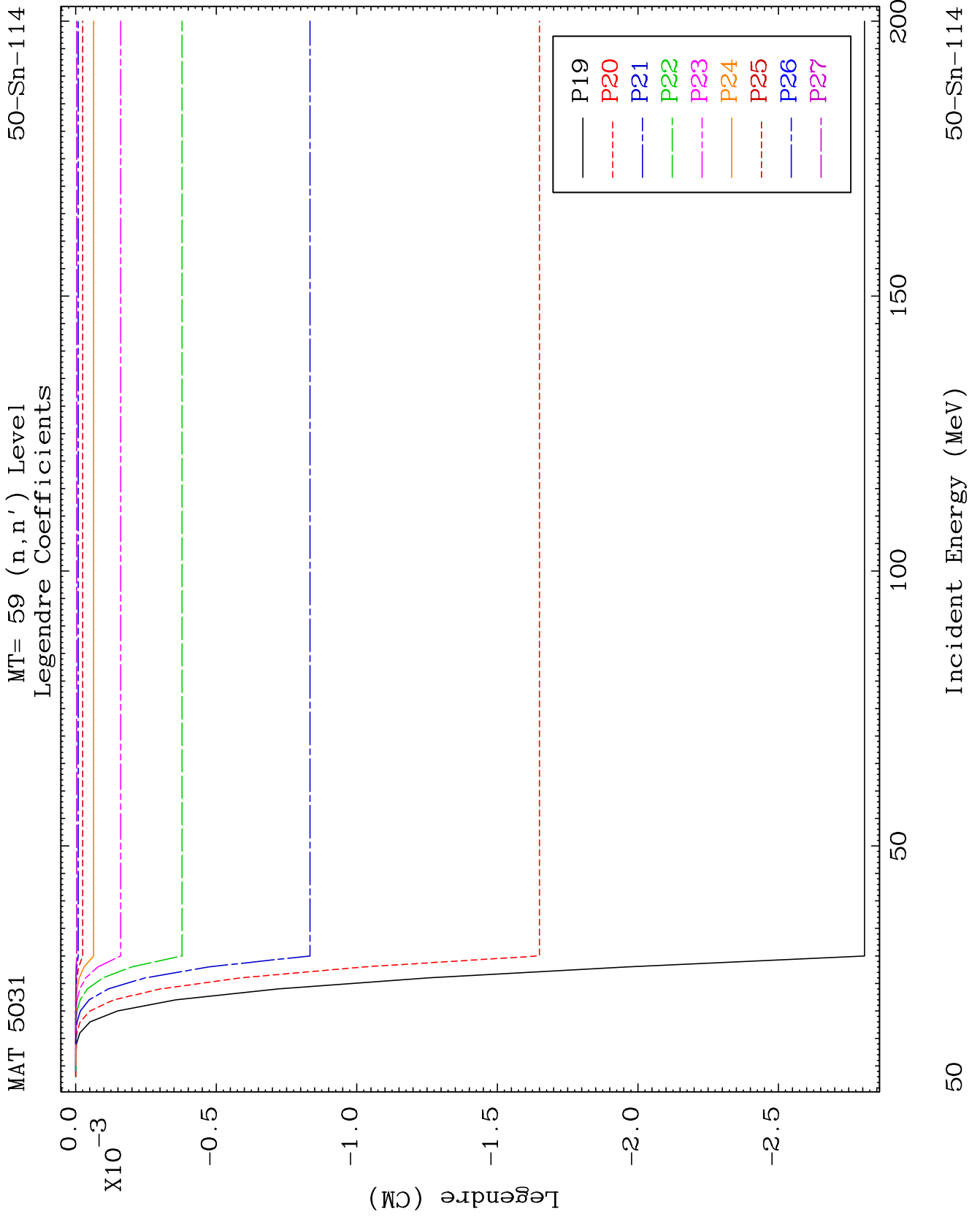


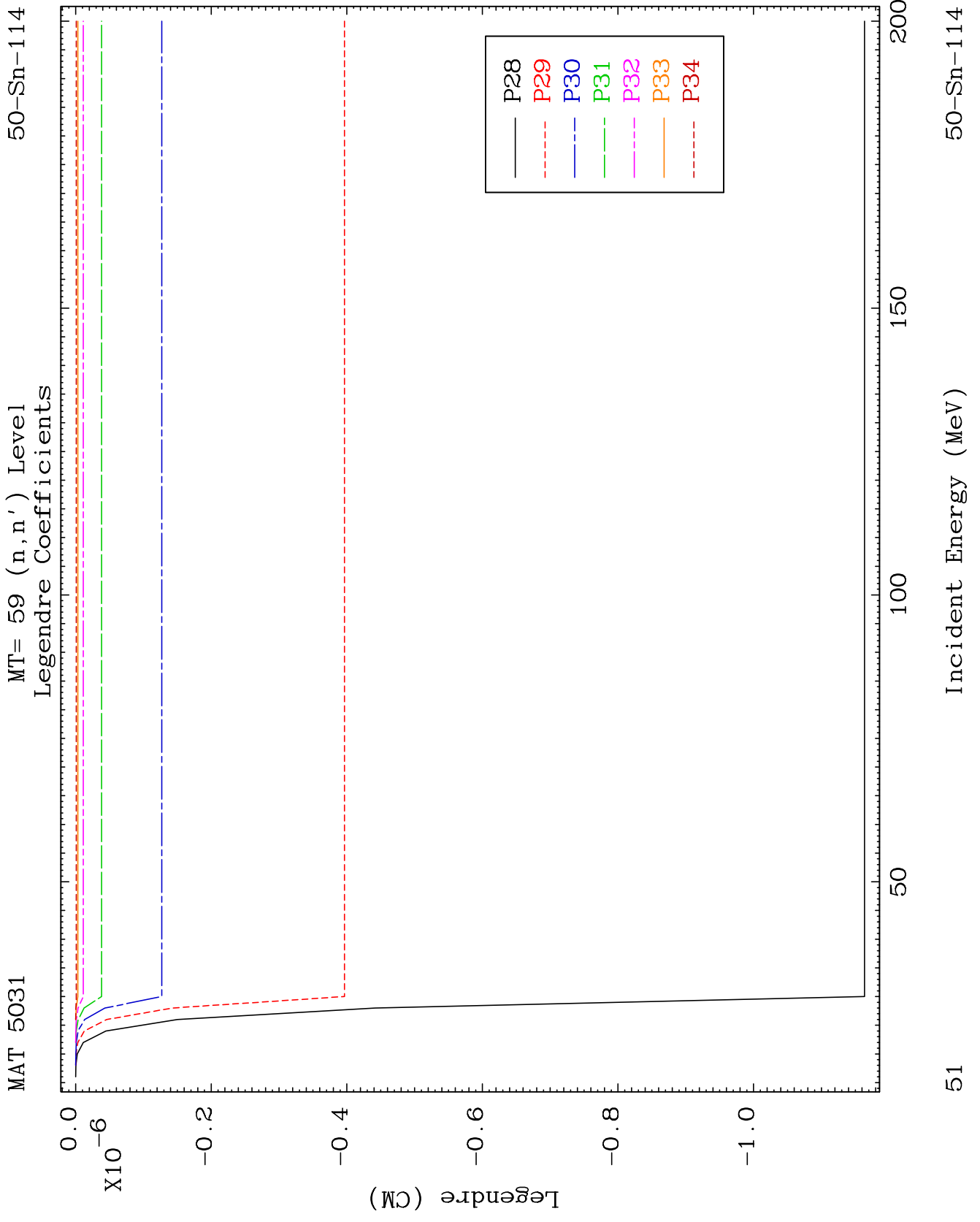
48

Incident Energy (MeV)

50-Sn-114



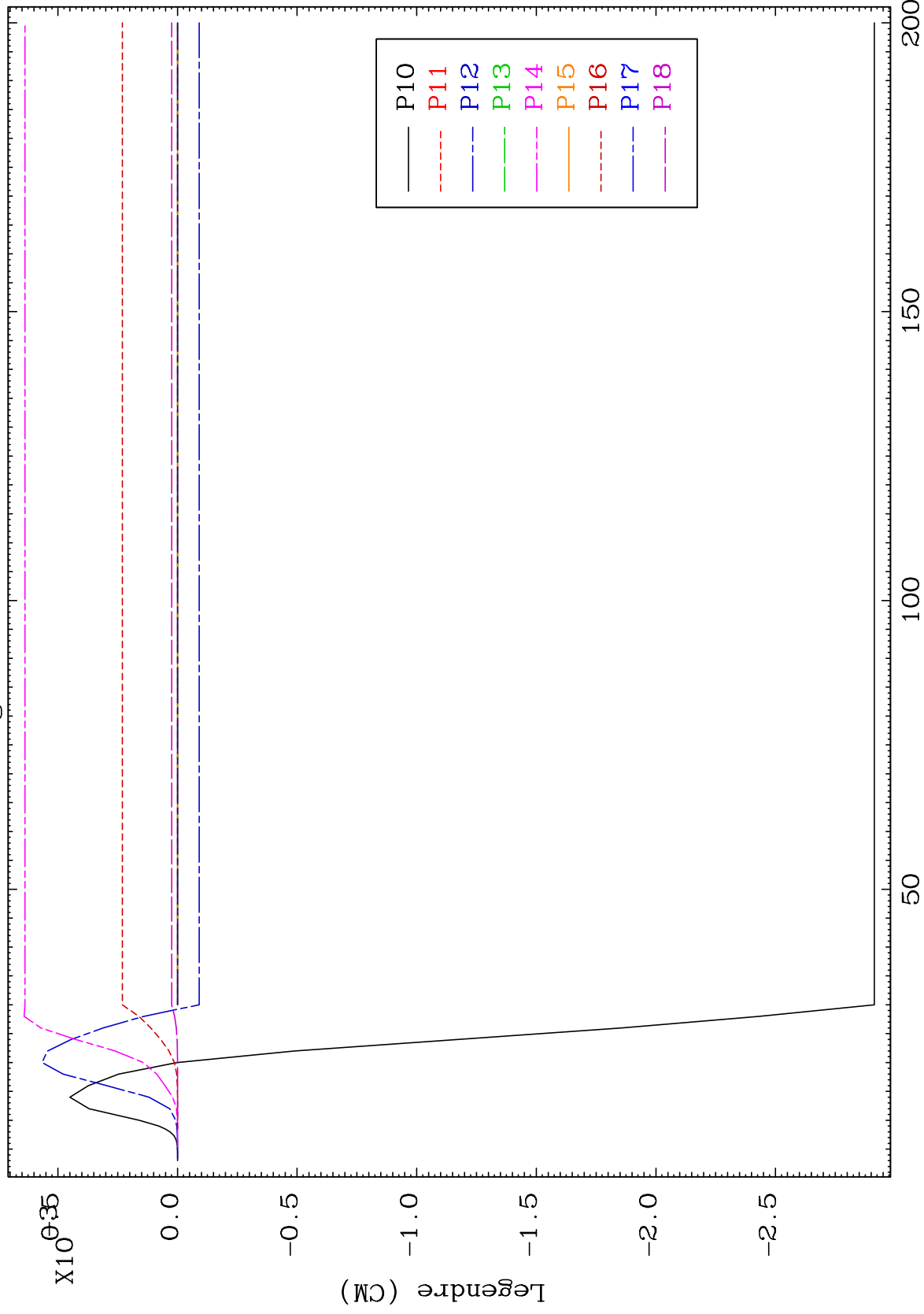




MAT 5031

50-Sn-114

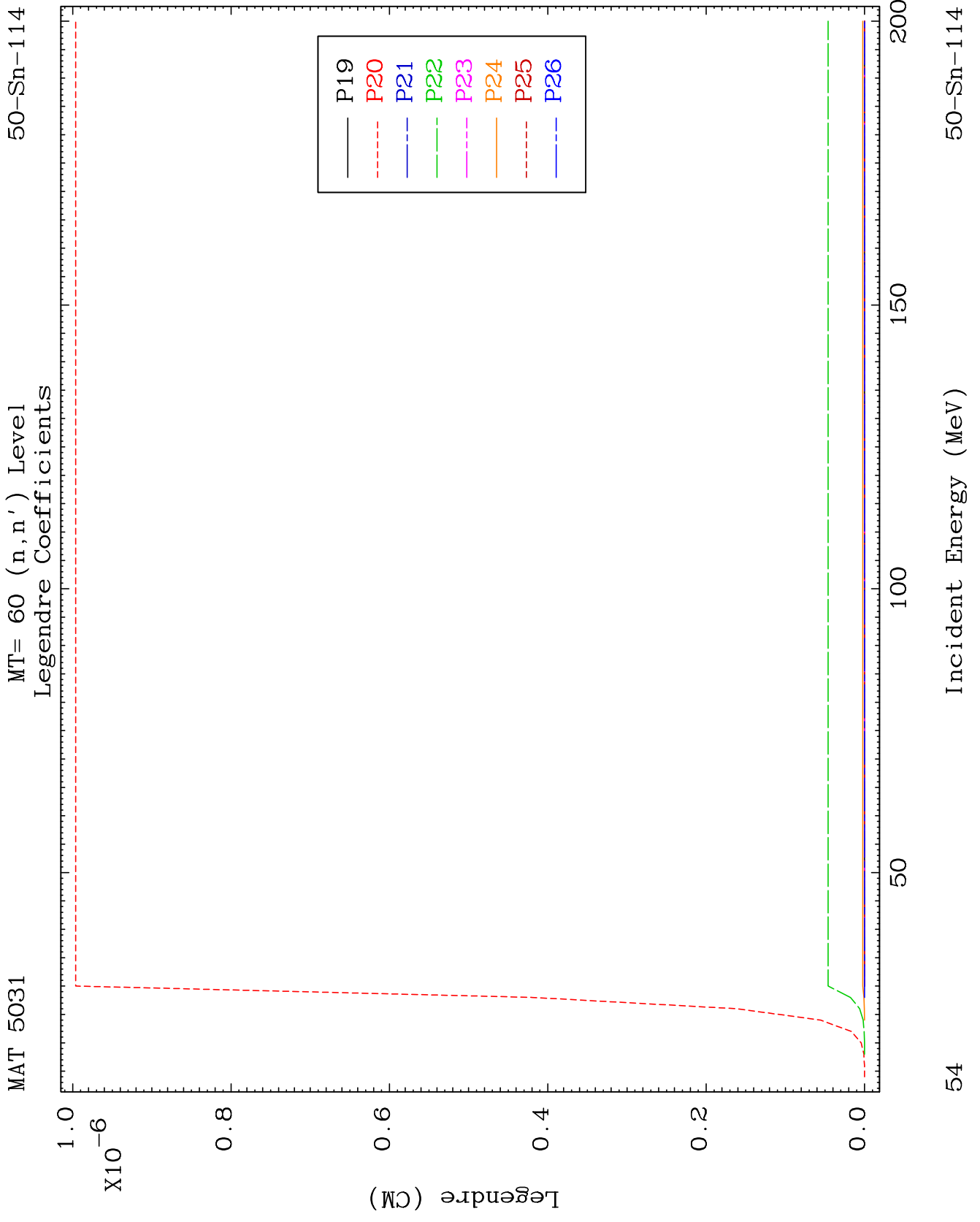
MT= 60 (n,n') Level
Legendre Coefficients

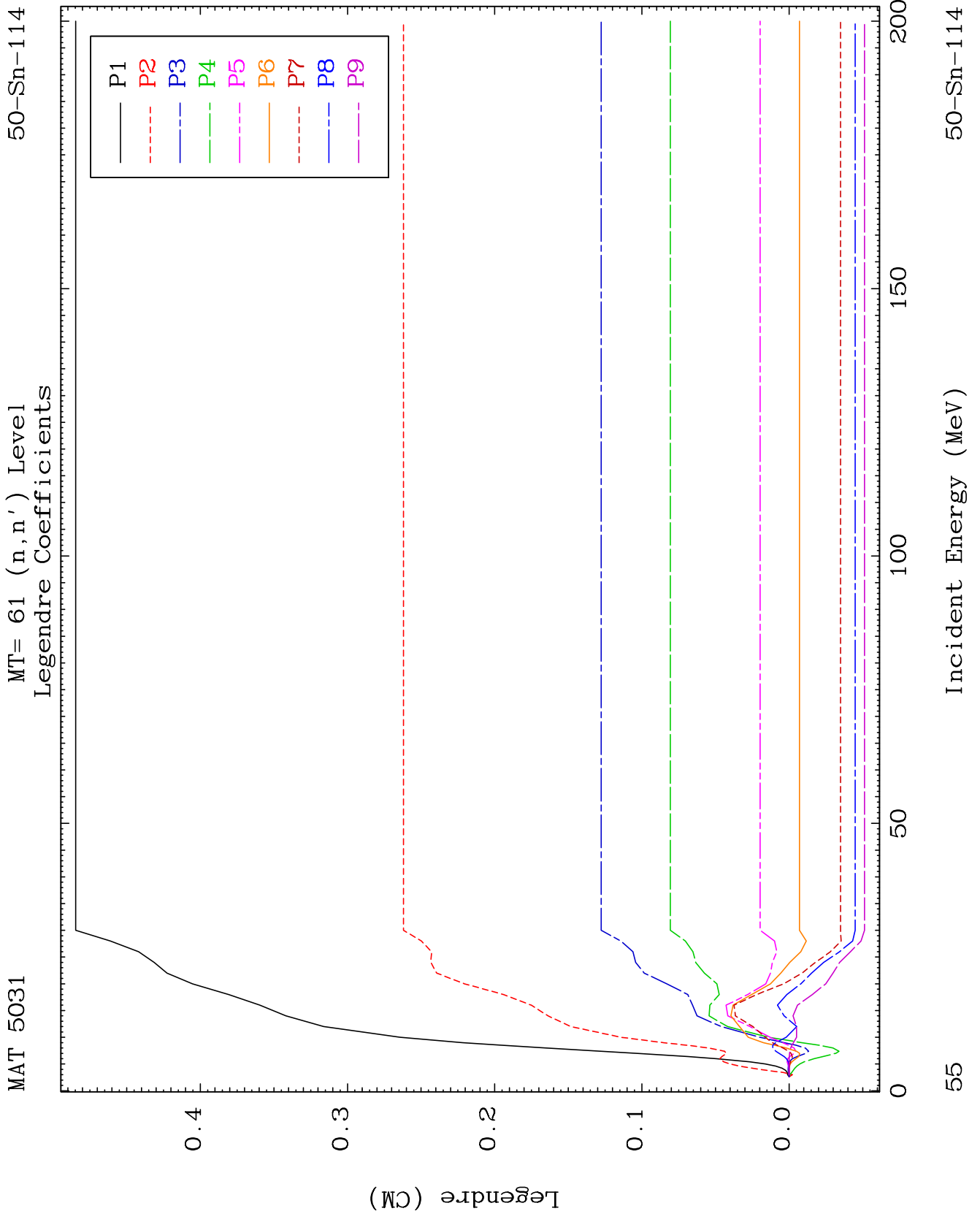


53

Incident Energy (MeV)

50-Sn-114

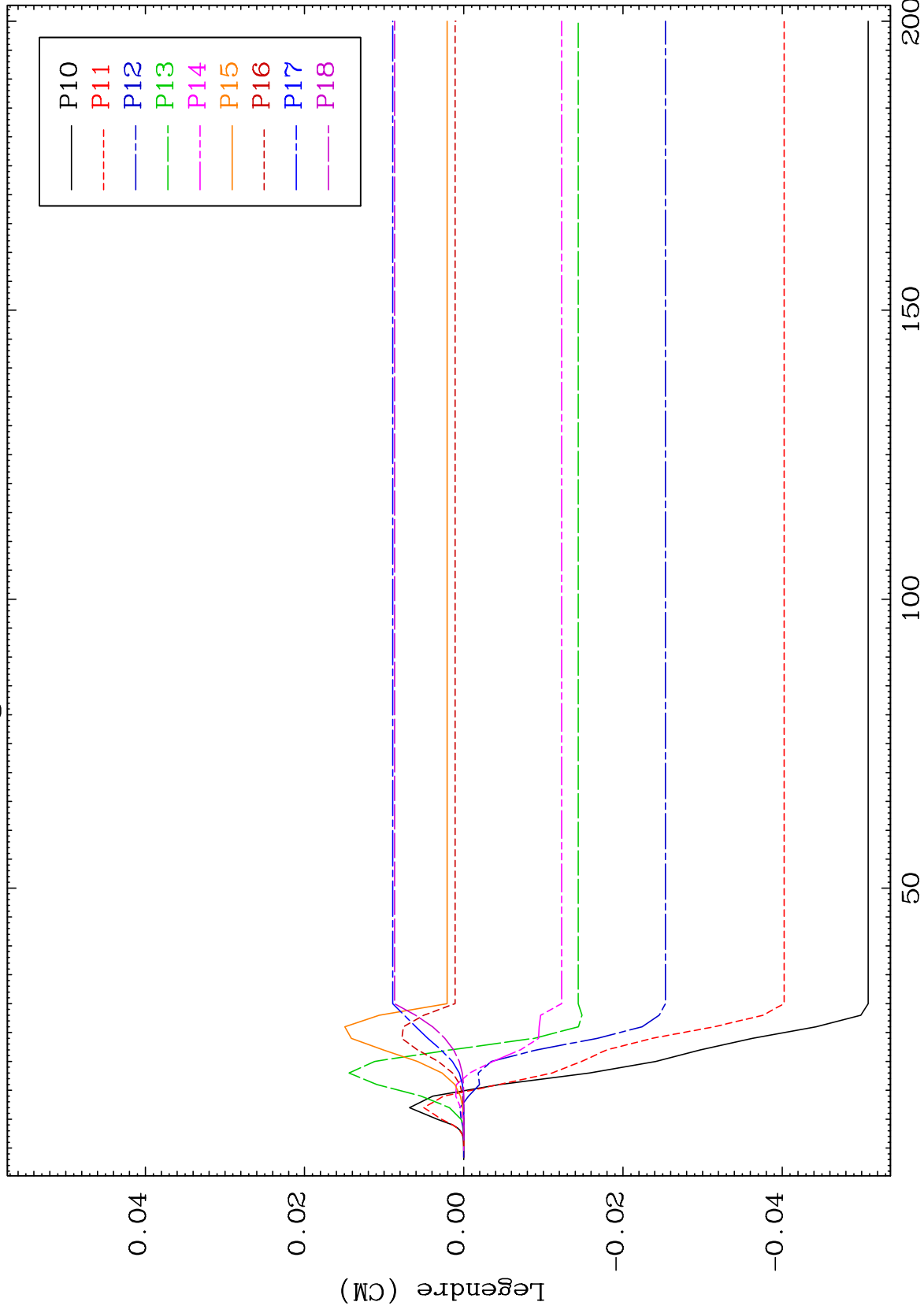




MAT 5031

MT= 61 (n,n') Level
Legendre Coefficients

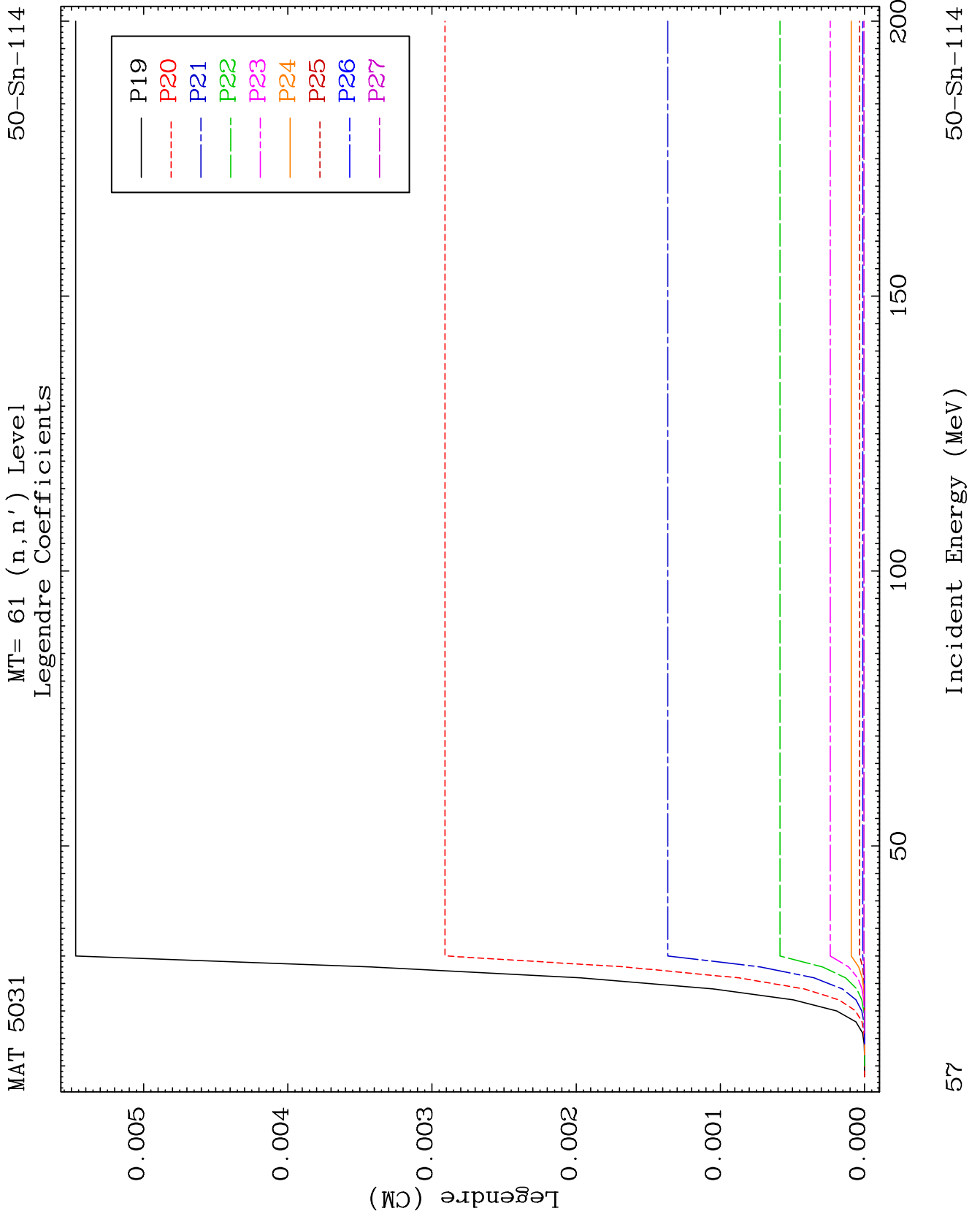
50-Sn-114



56

Incident Energy (MeV)

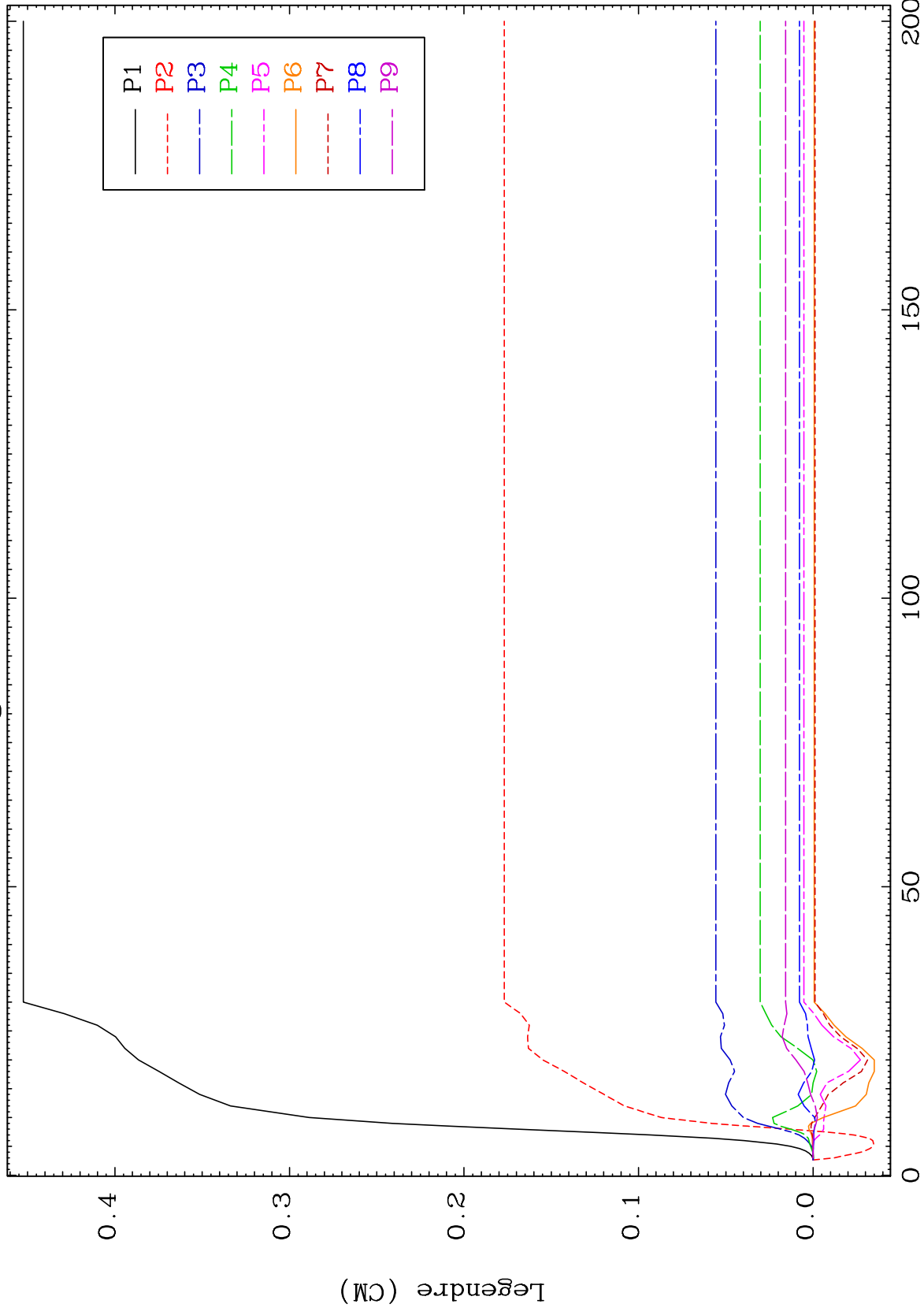
50-Sn-114



MAT 5031

MT= 62 (n,n') Level
Legendre Coefficients

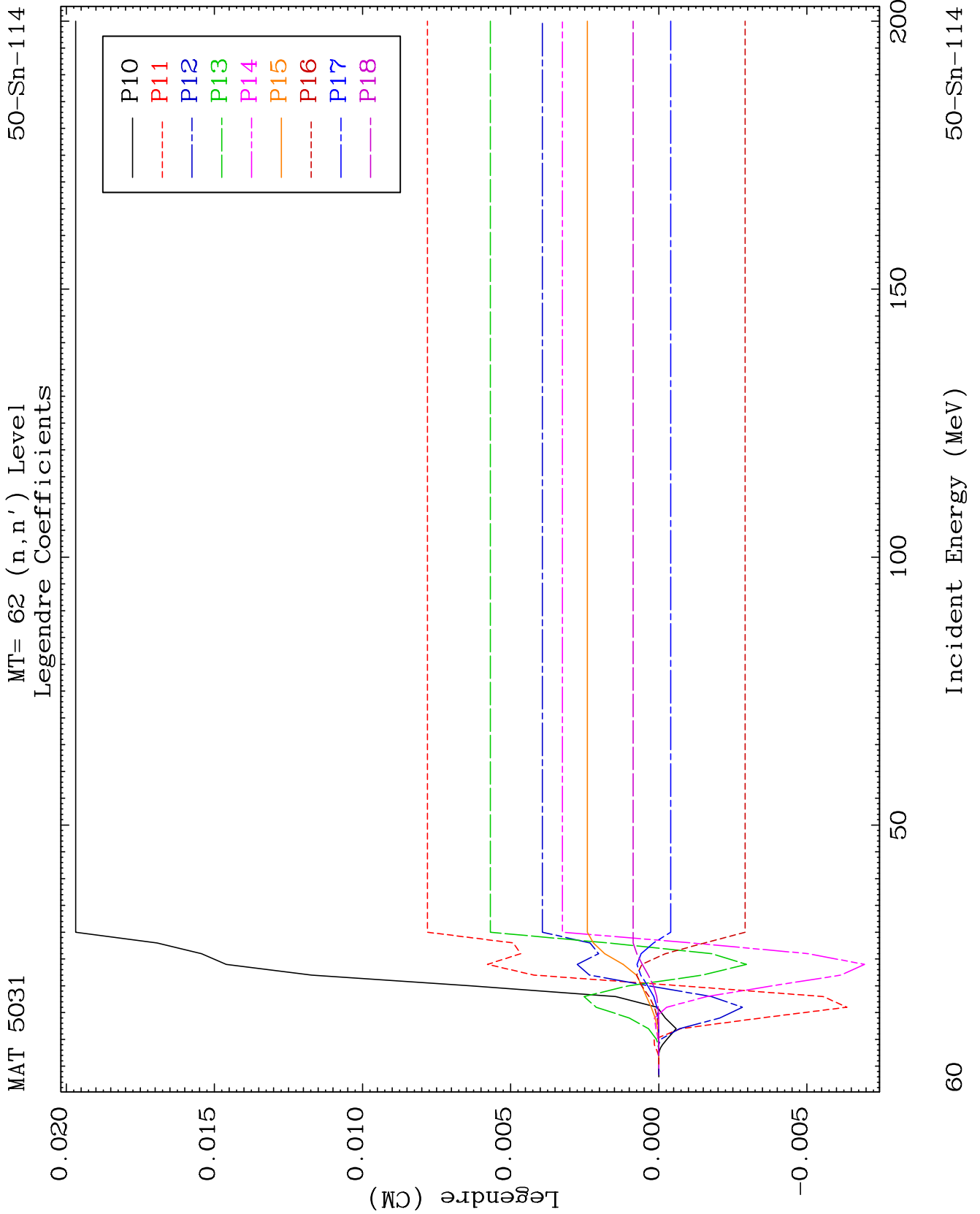
50-Sn-114



59

Incident Energy (MeV)

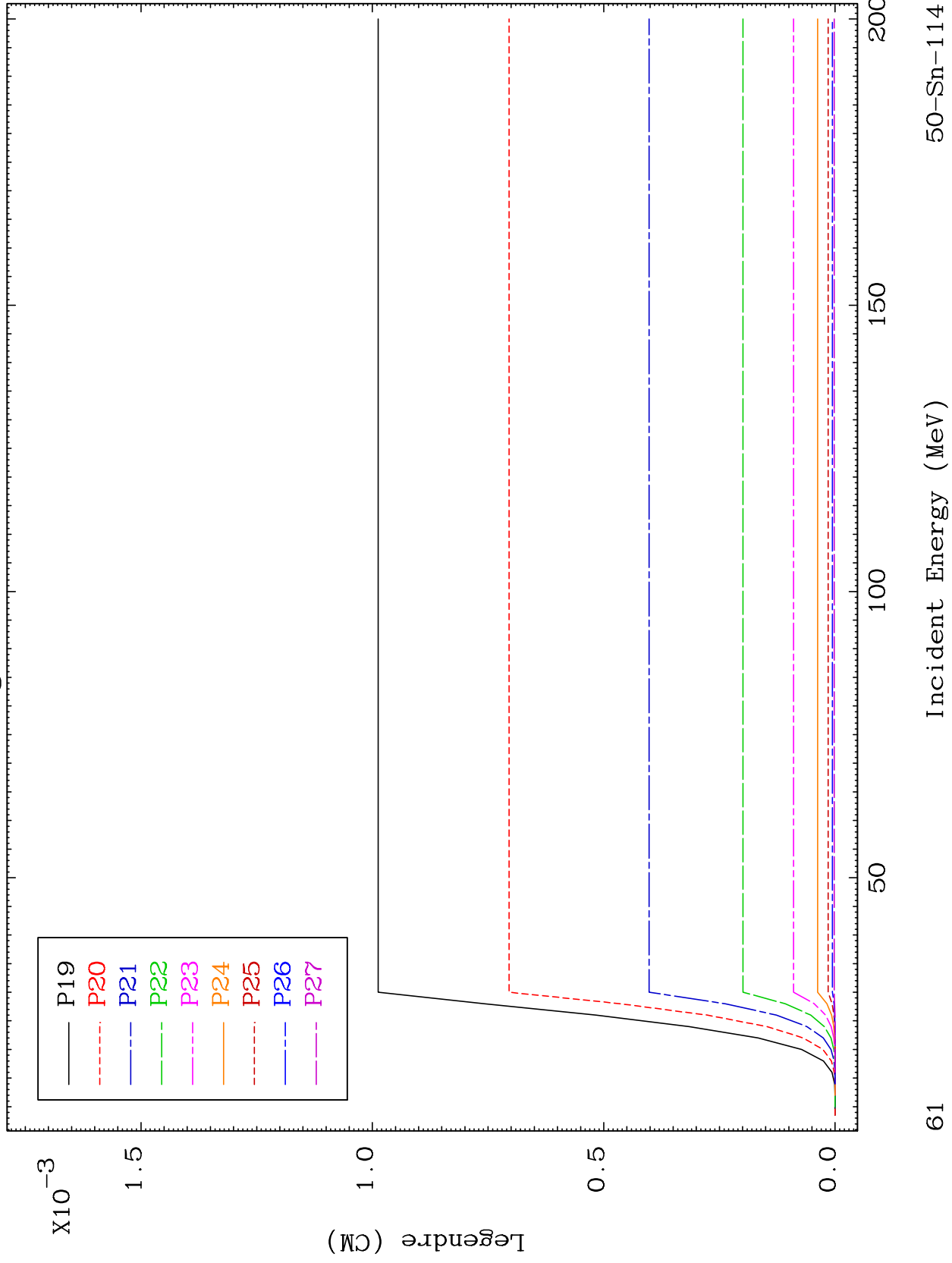
50-Sn-114



MAT 5031

MT= 62 (n,n') Level
Legendre Coefficients

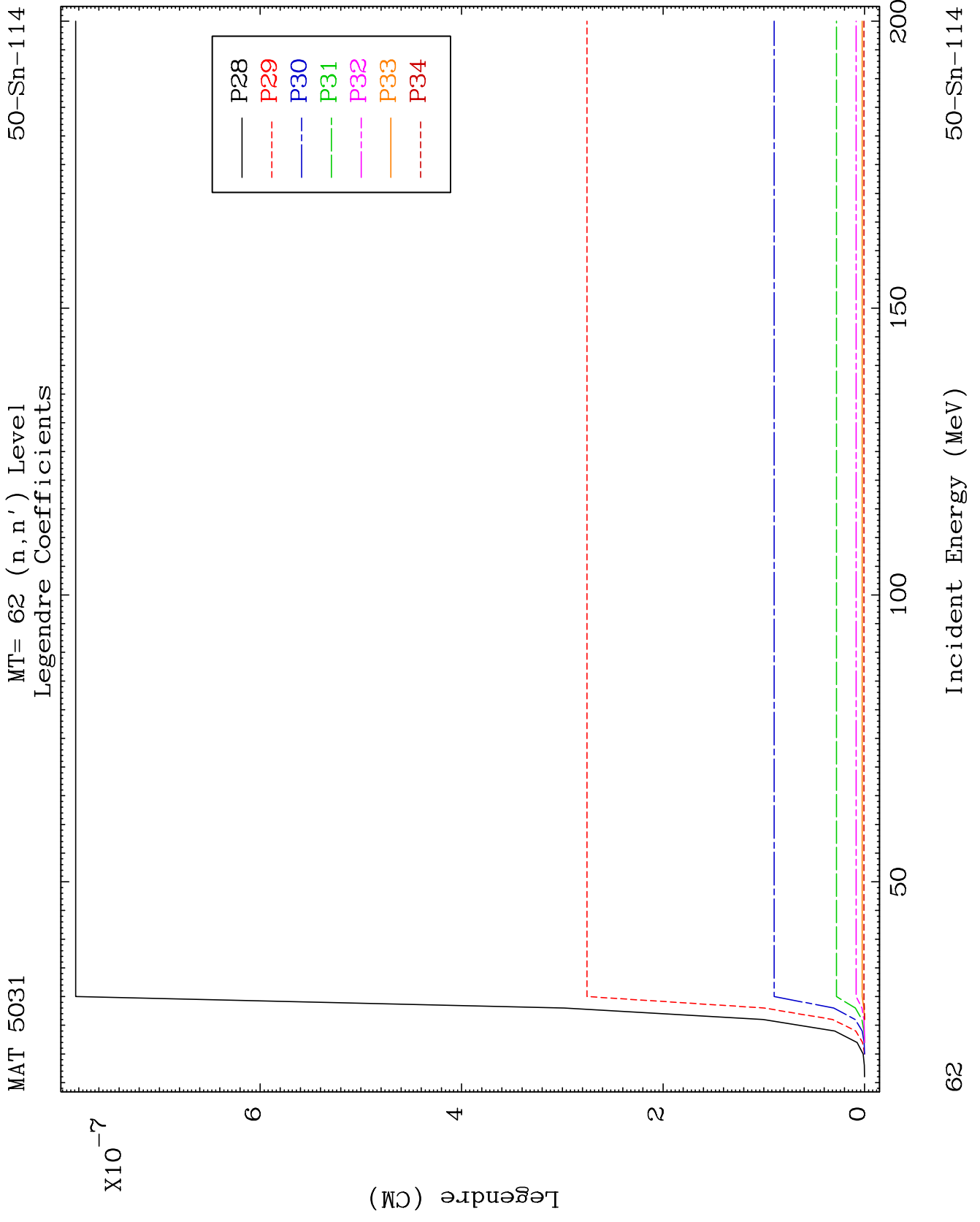
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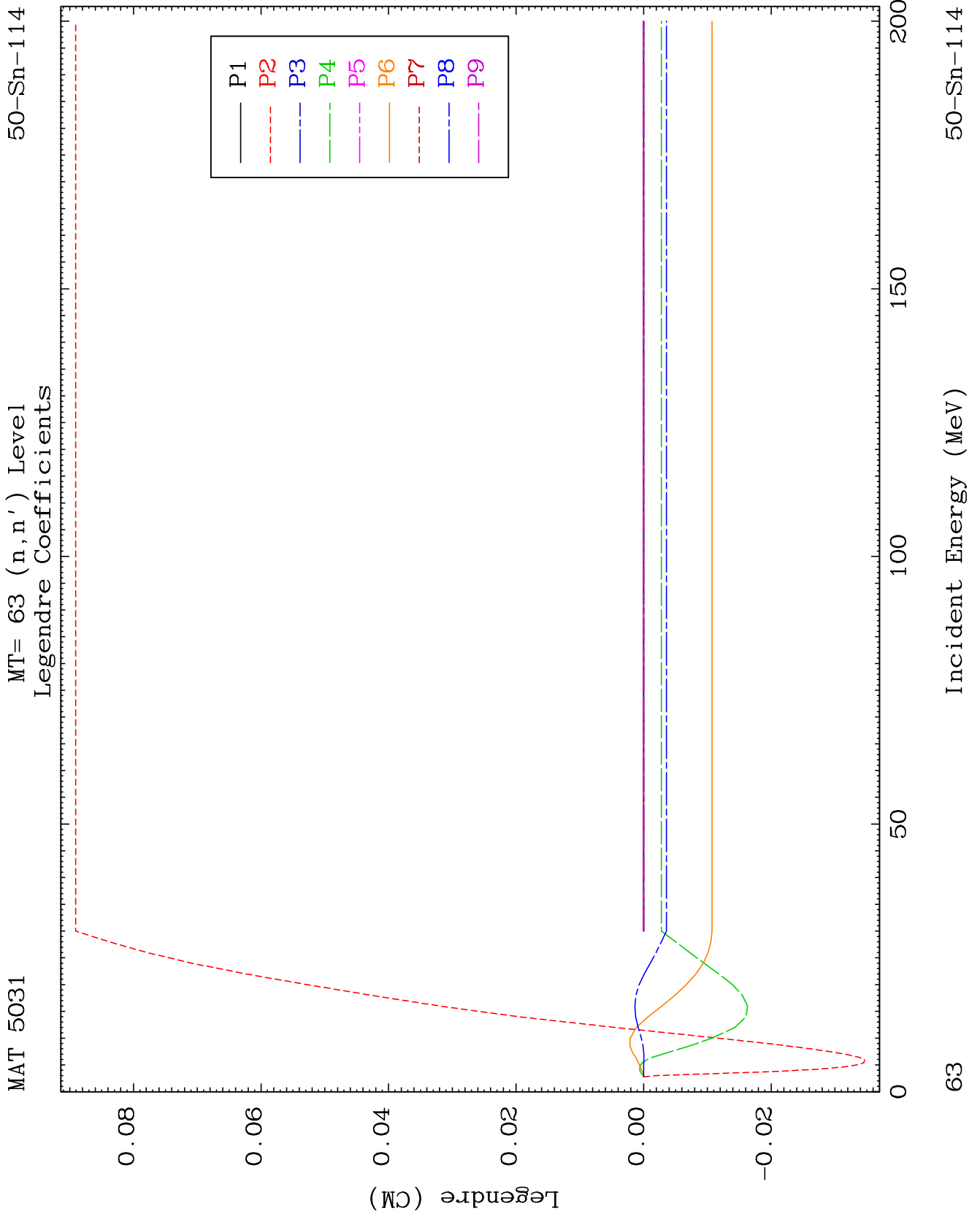


61

Incident Energy (MeV)

50-Sn-114

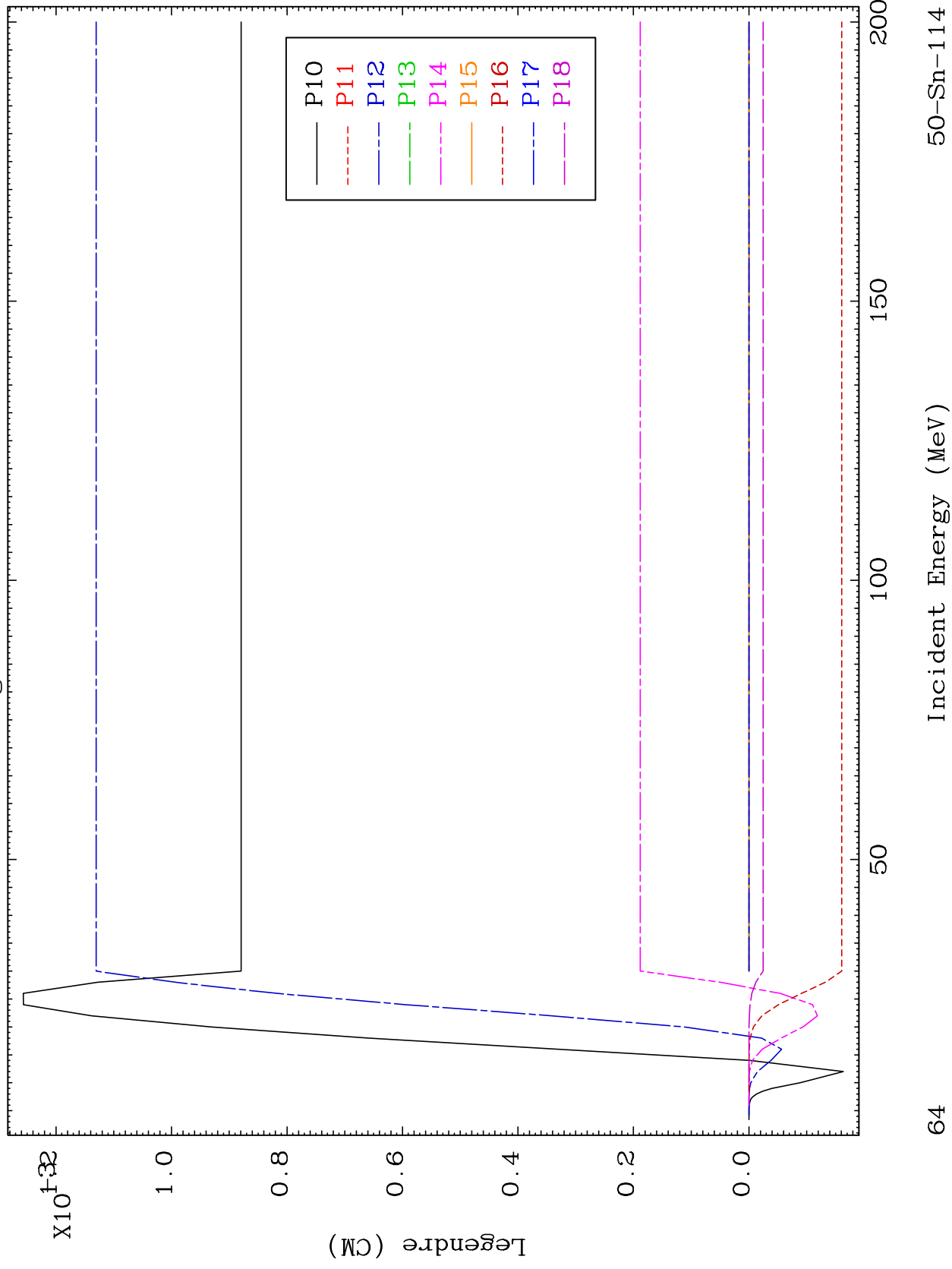




MAT 5031

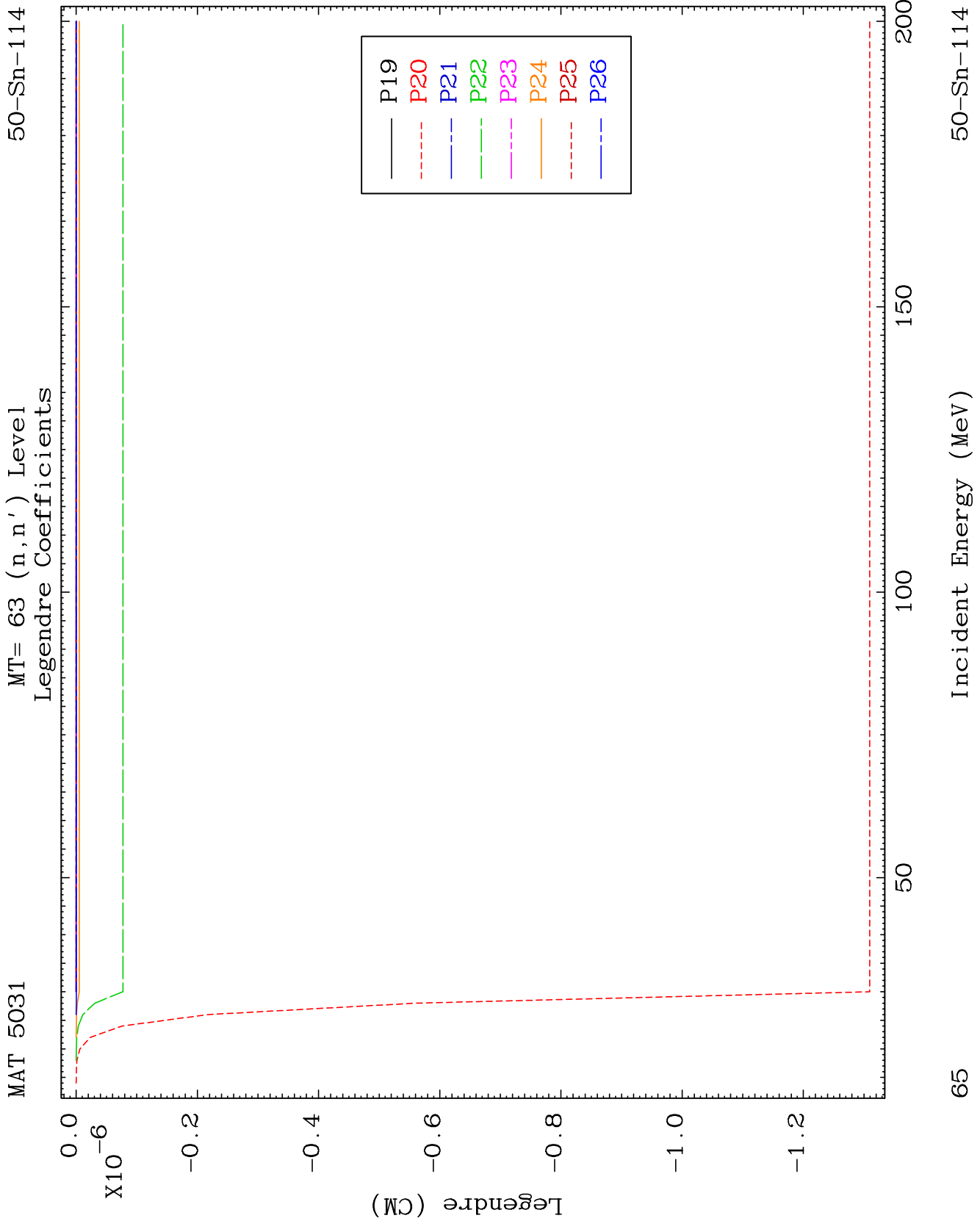
MT= 63 (n,n') Level
Legendre Coefficients

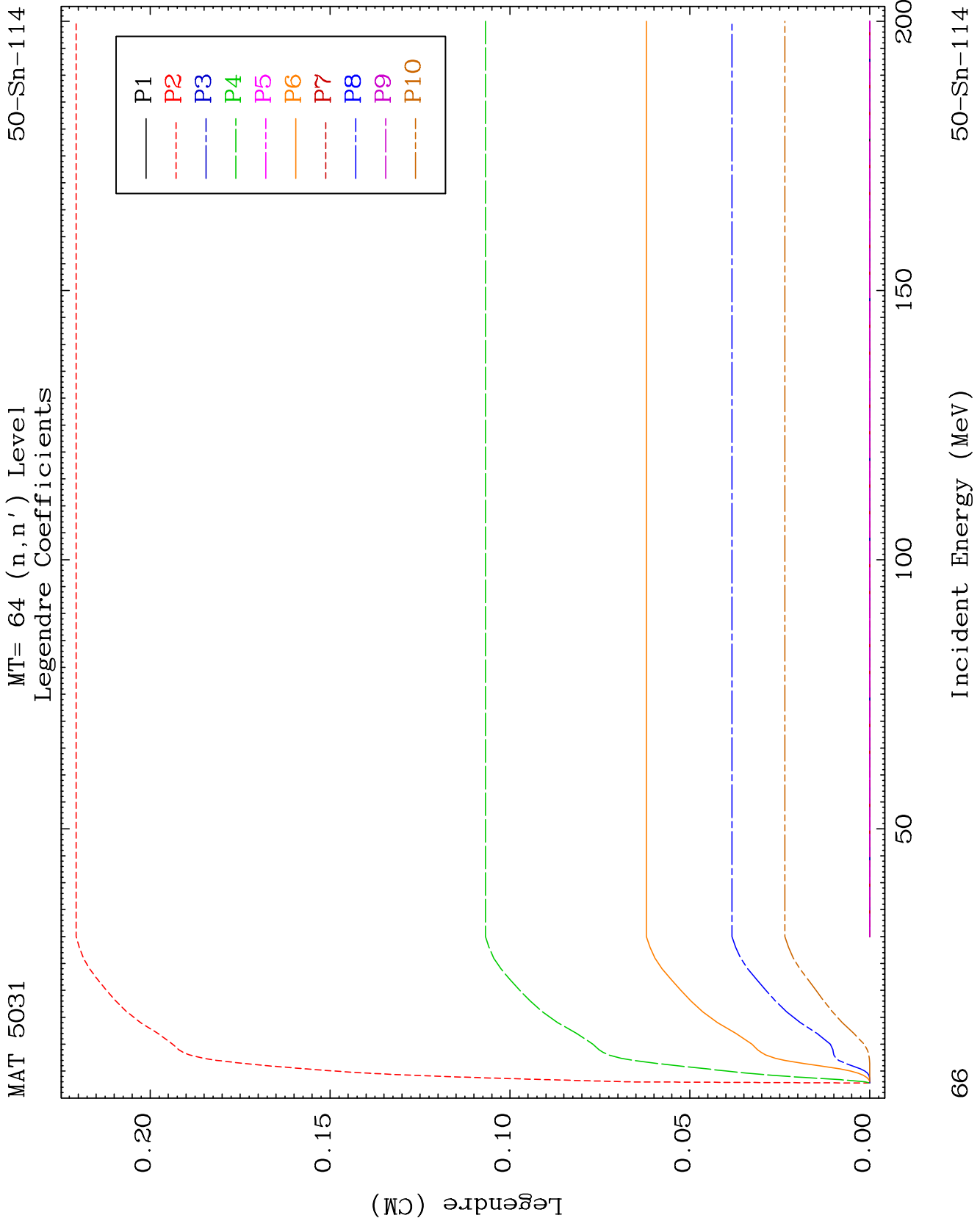
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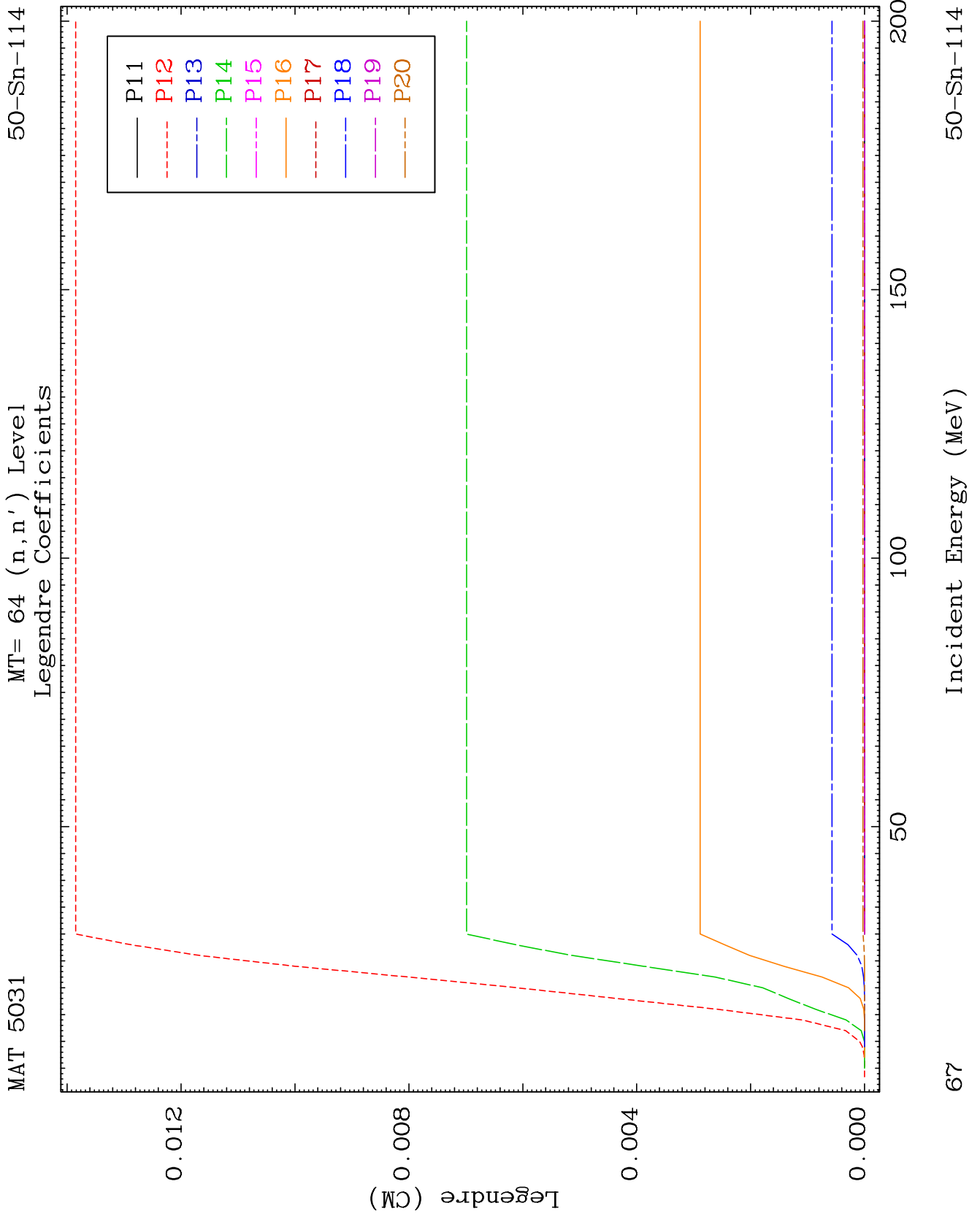


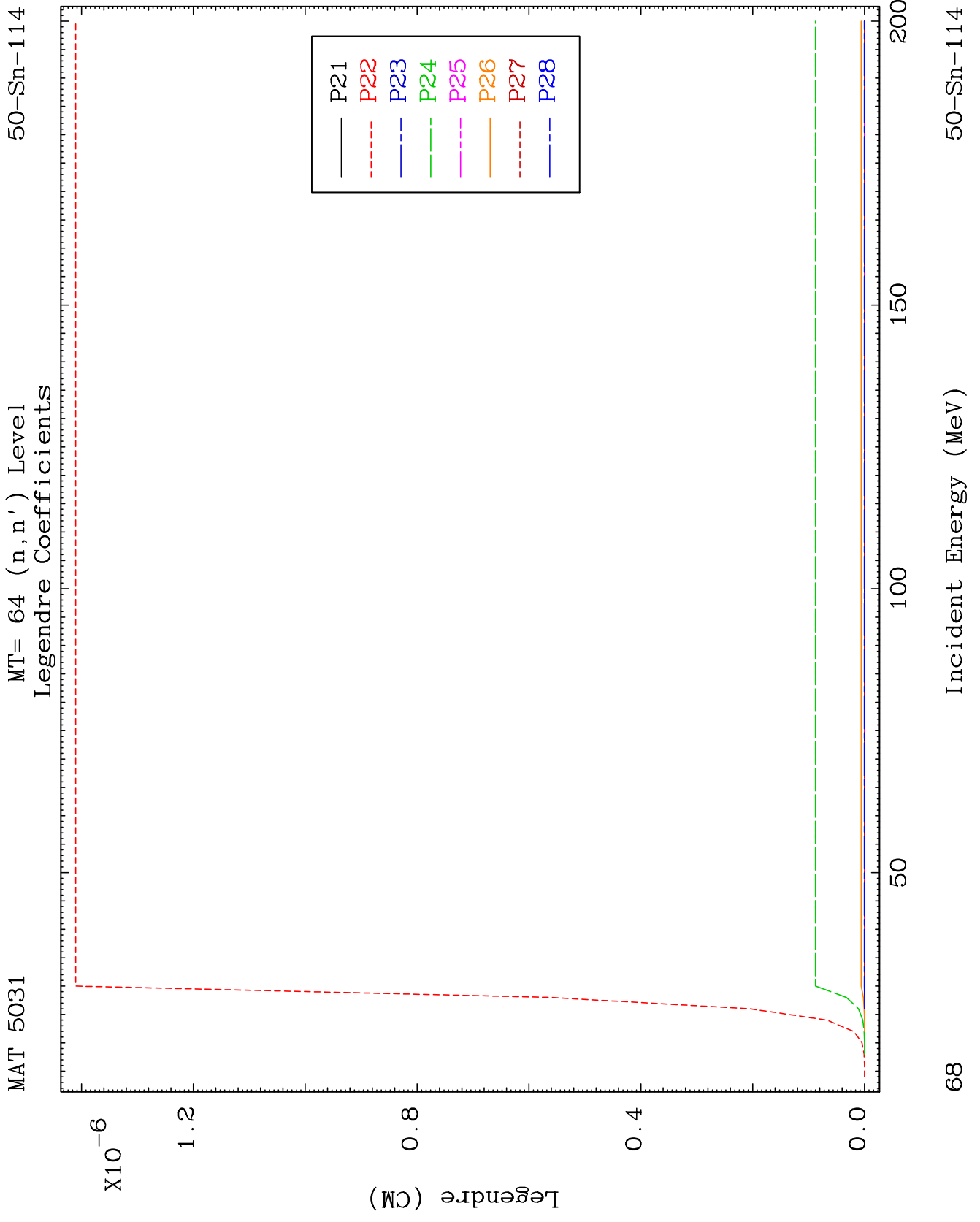
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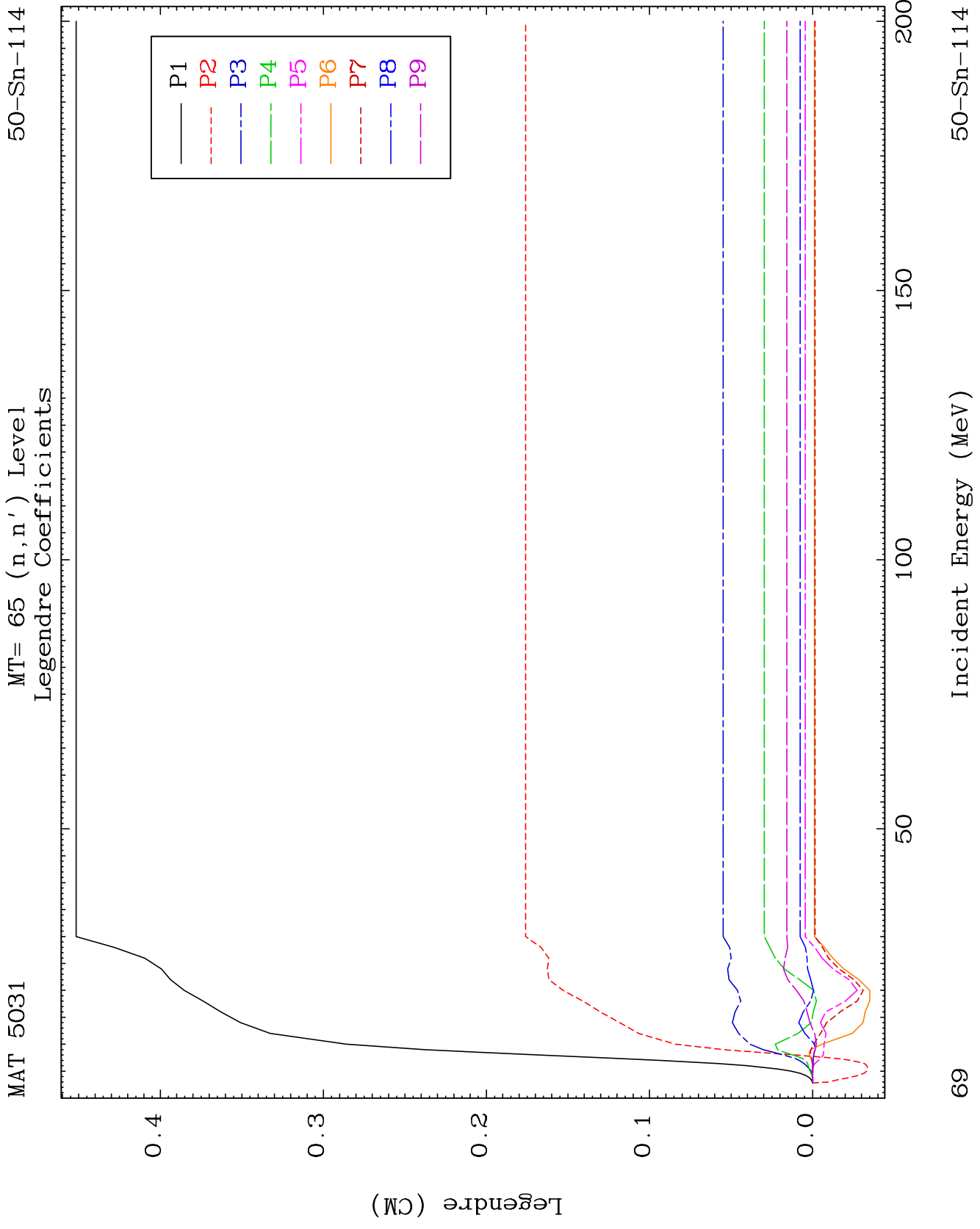
50-Sn-114







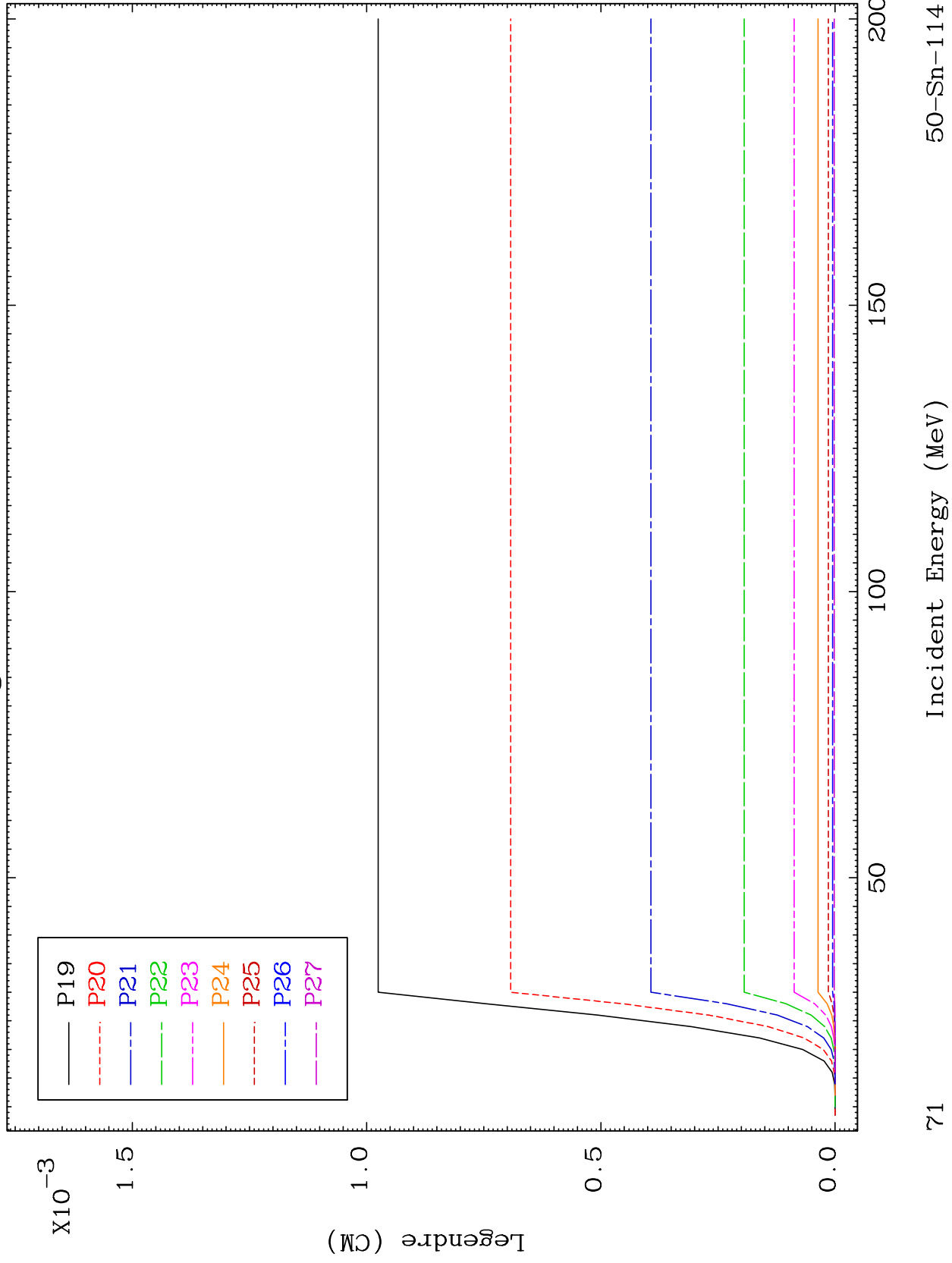




MAT 5031

MT= 65 (n,n') Level
Legendre Coefficients

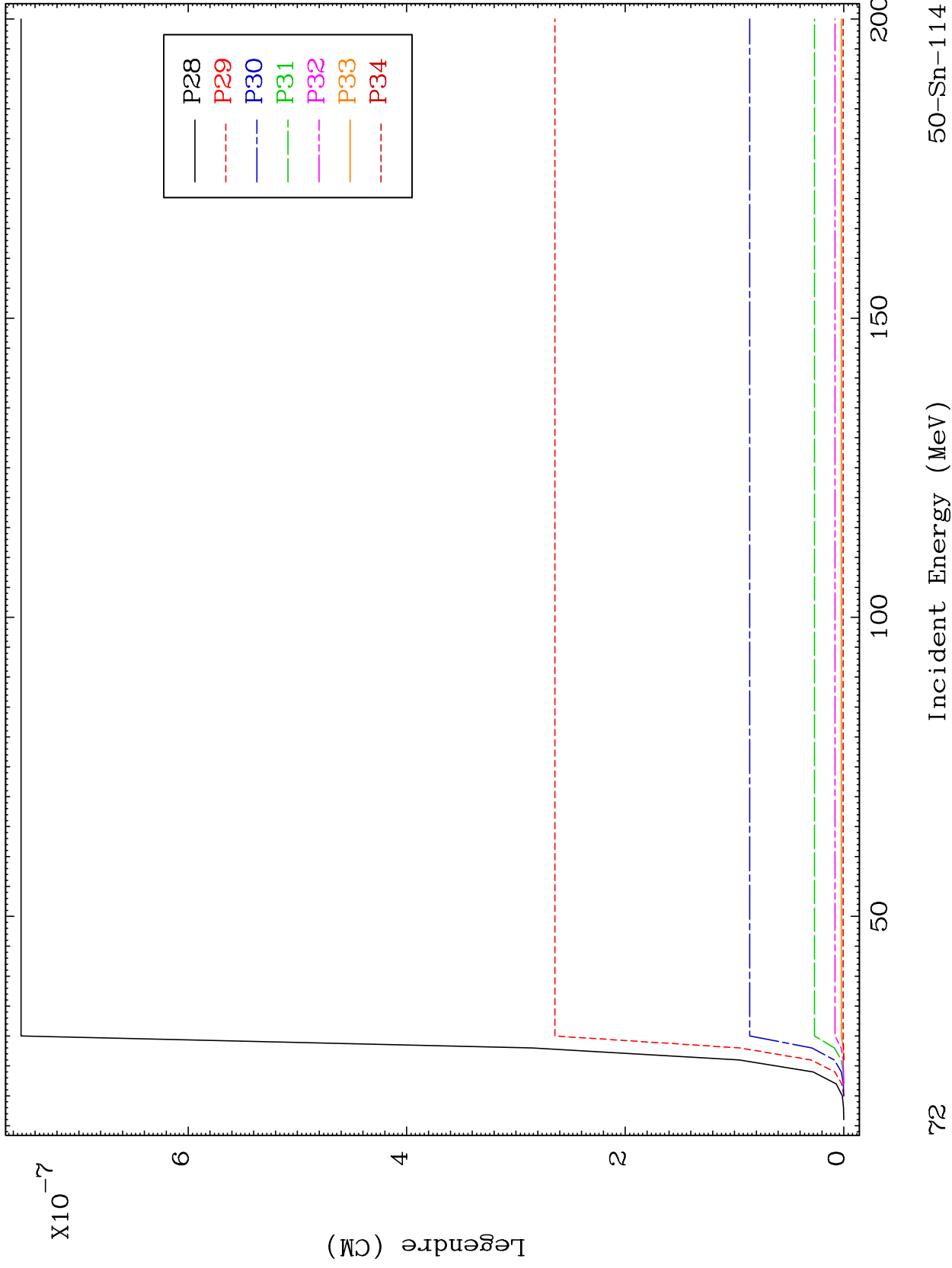
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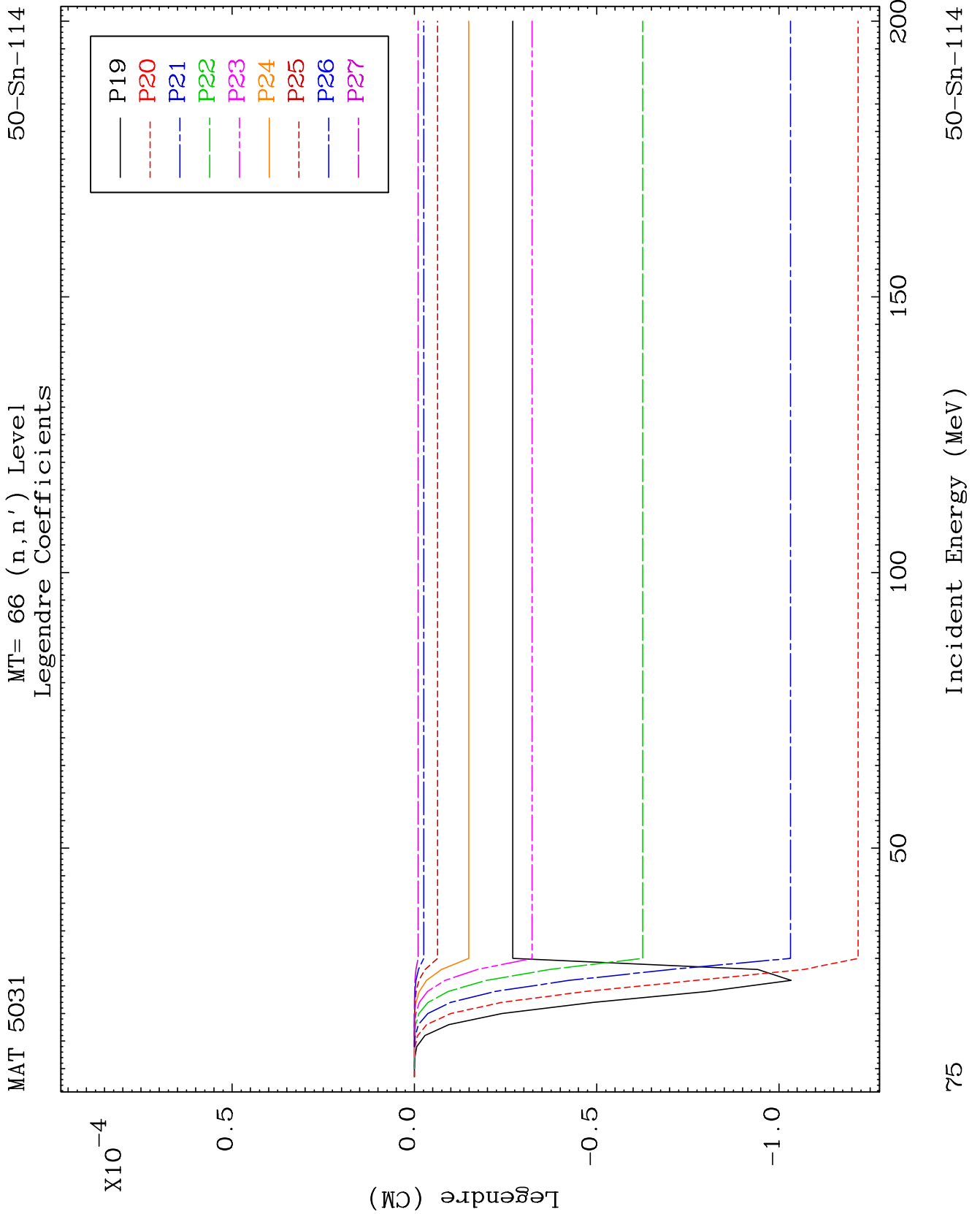


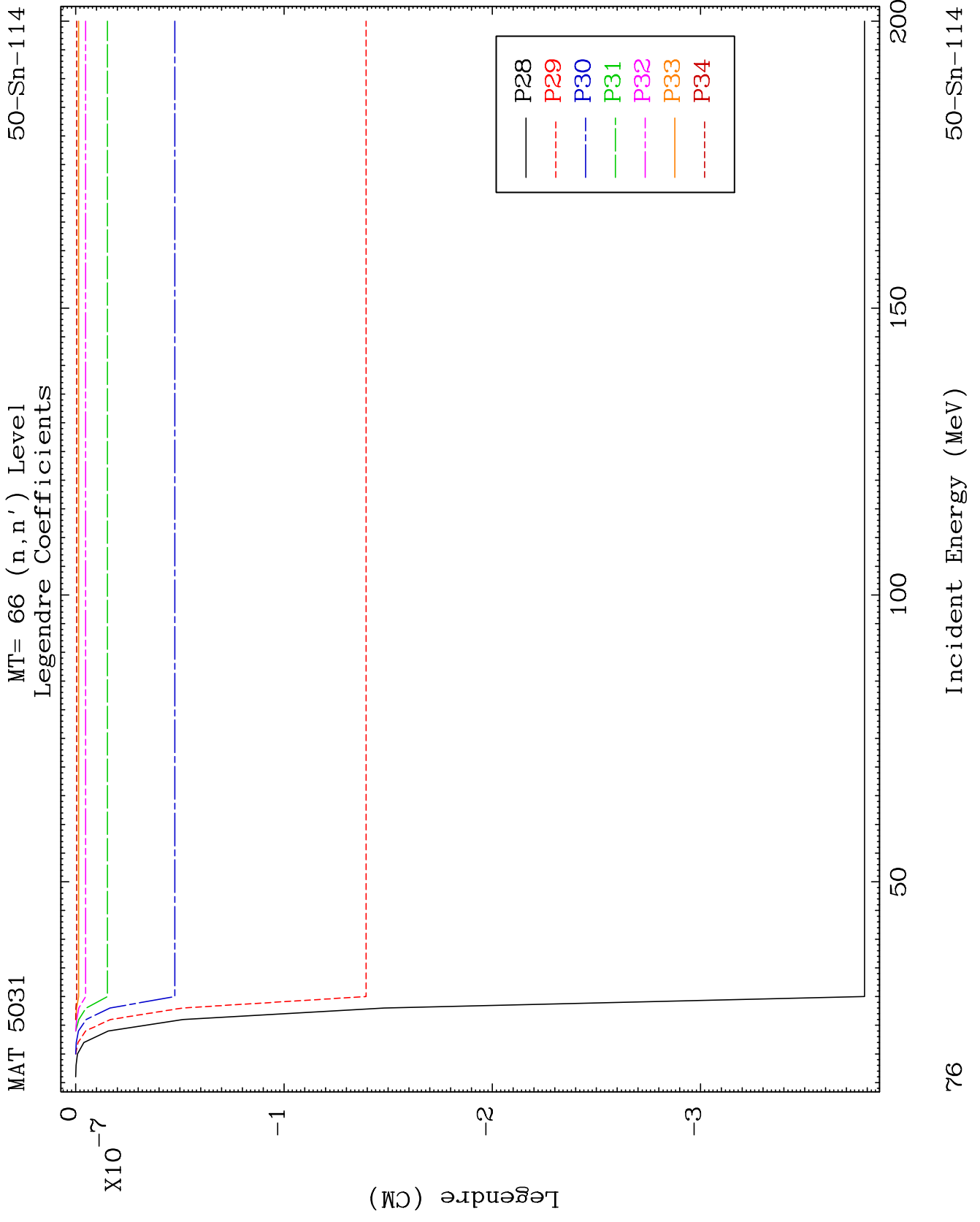
MAT 5031

MT= 65 (n,n') Level
Legendre Coefficients

50-Sn-114



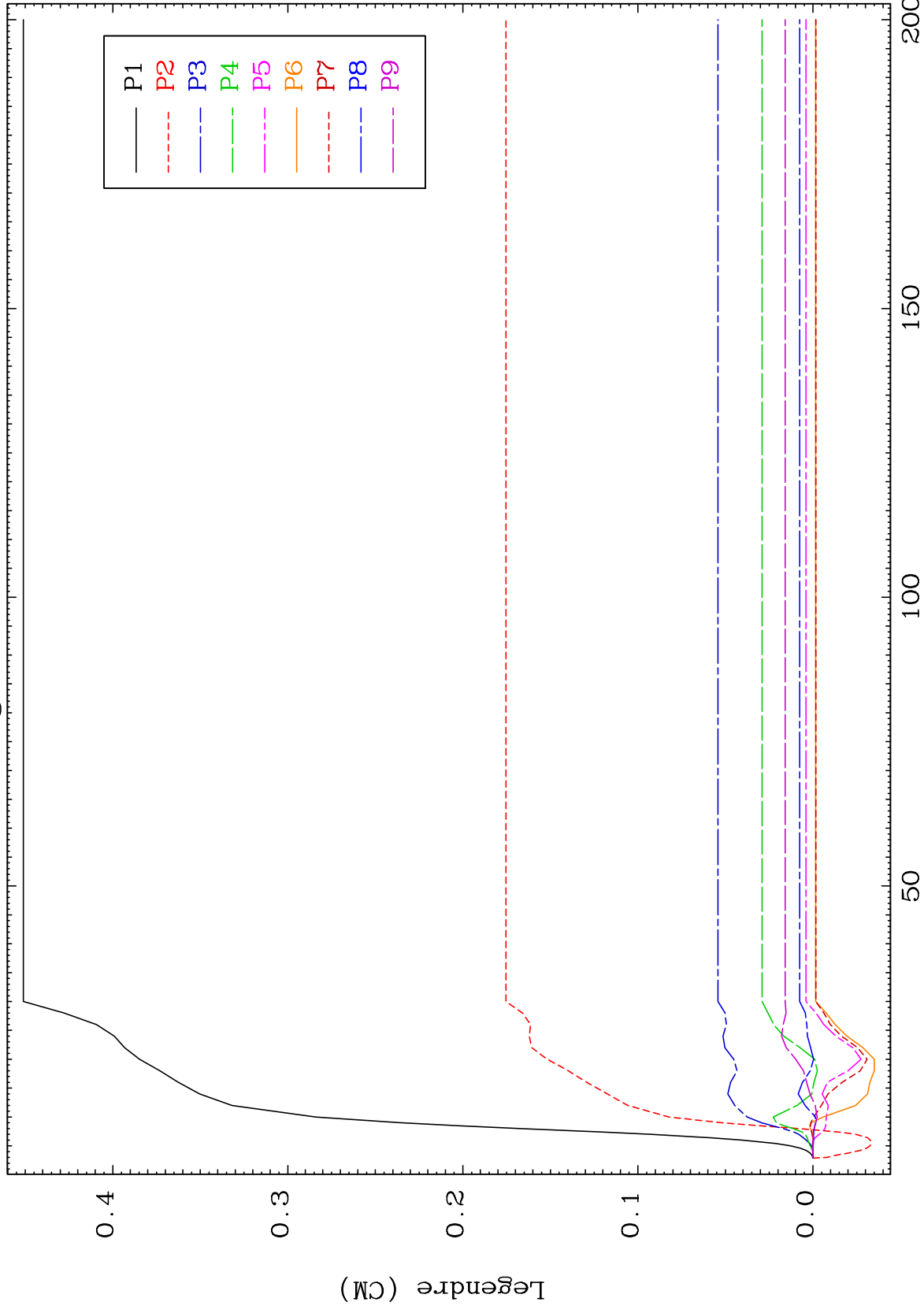


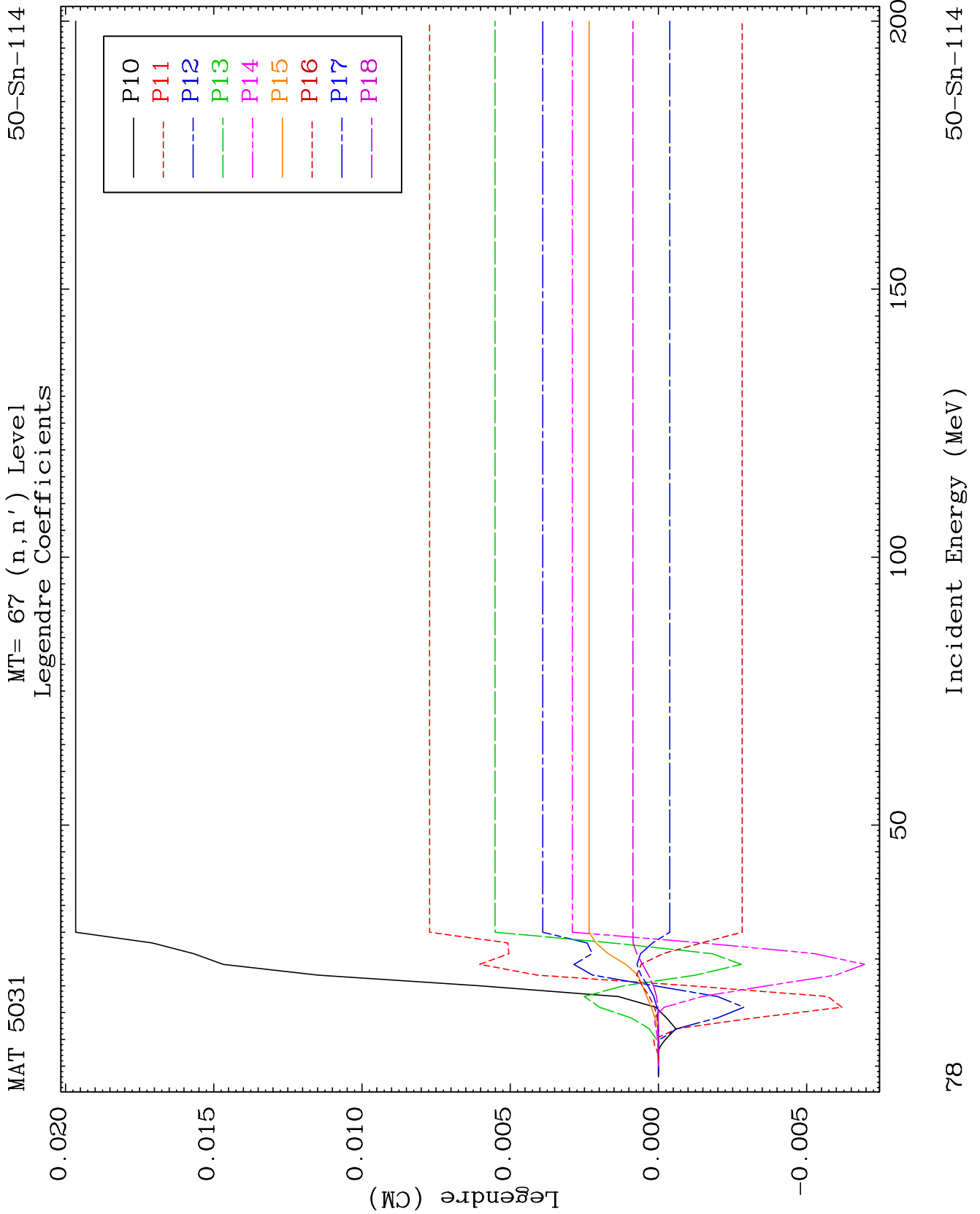


MAT 5031

MT= 67 (n,n') Level
Legendre Coefficients

50-Sn-114

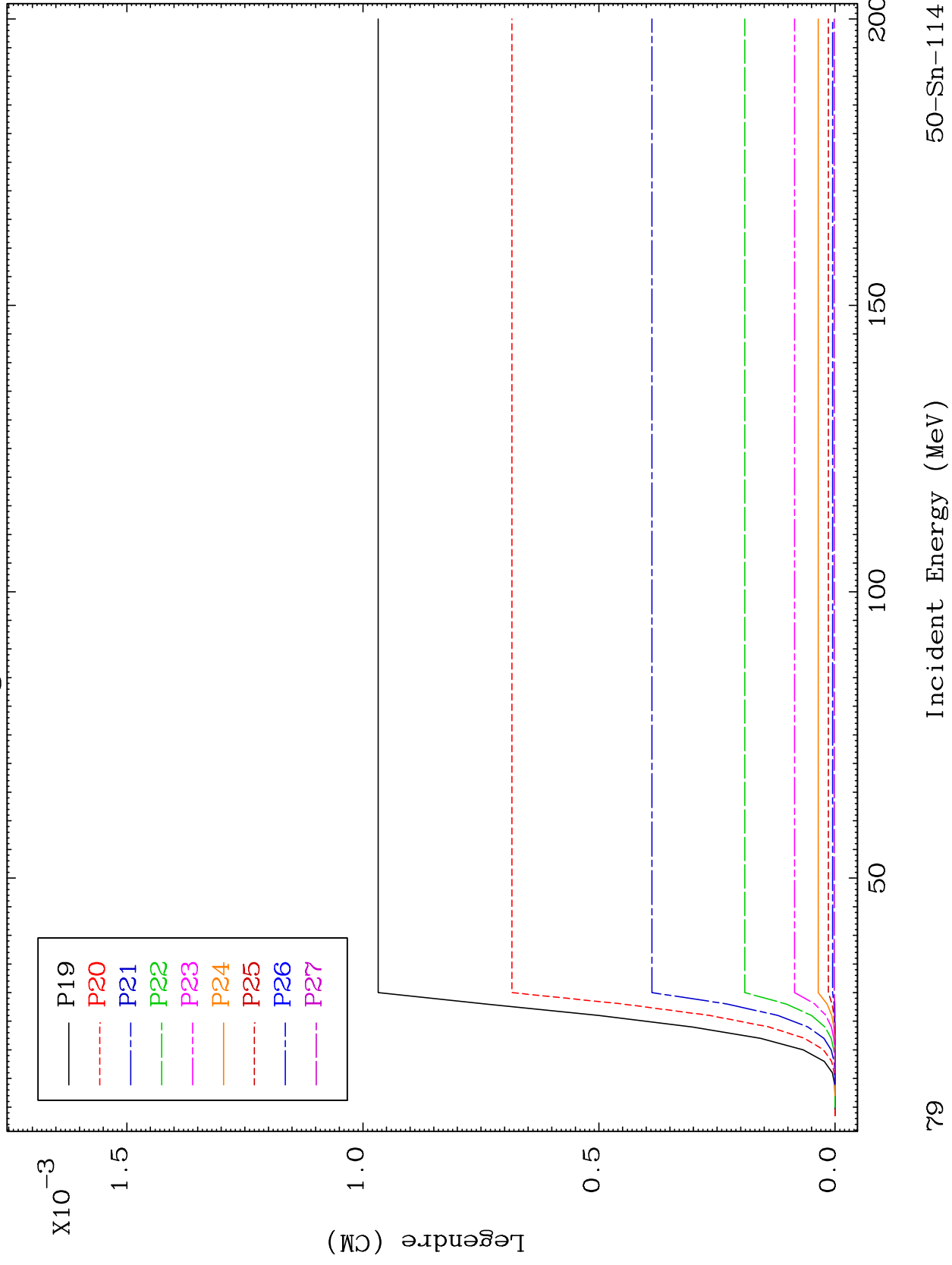


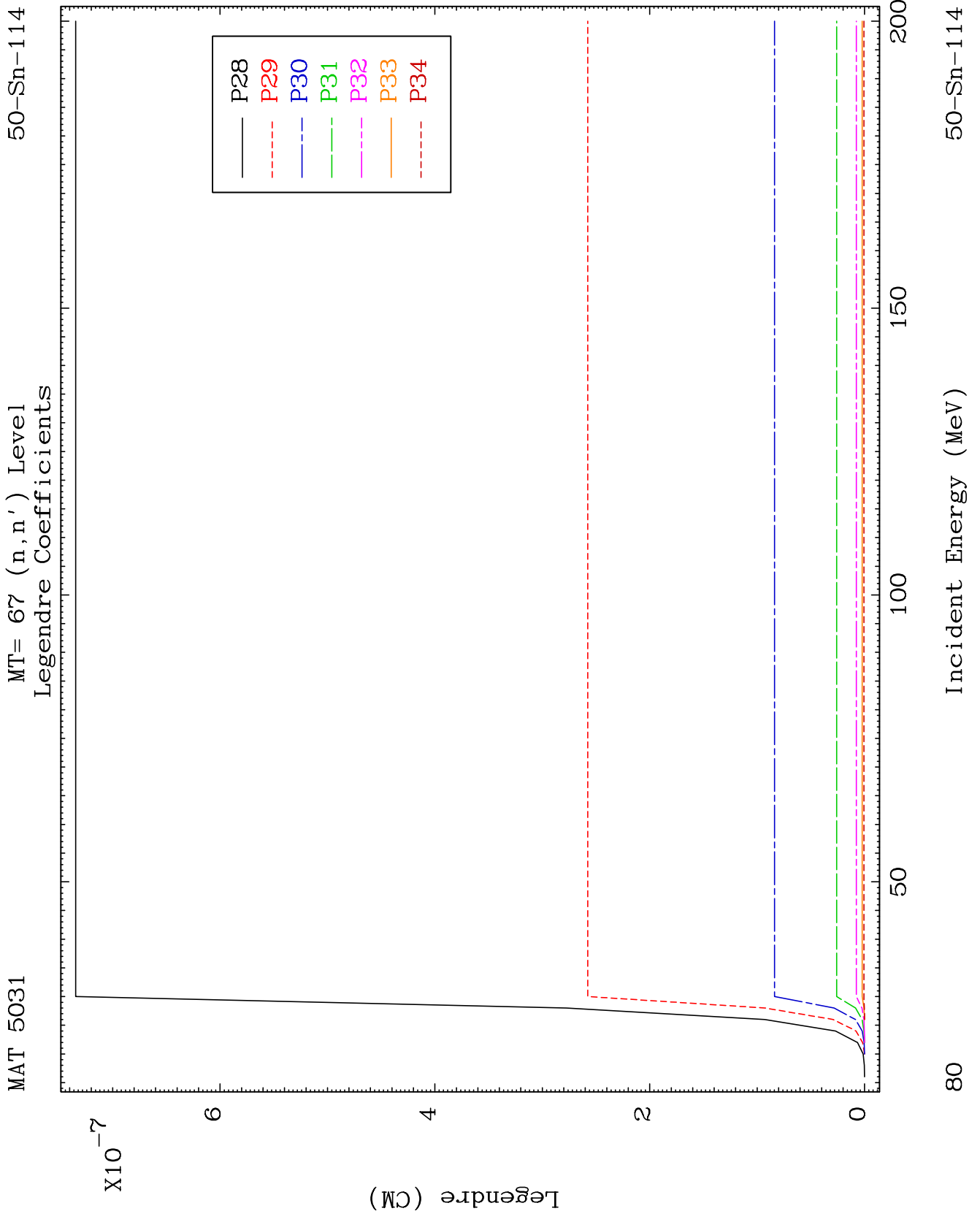


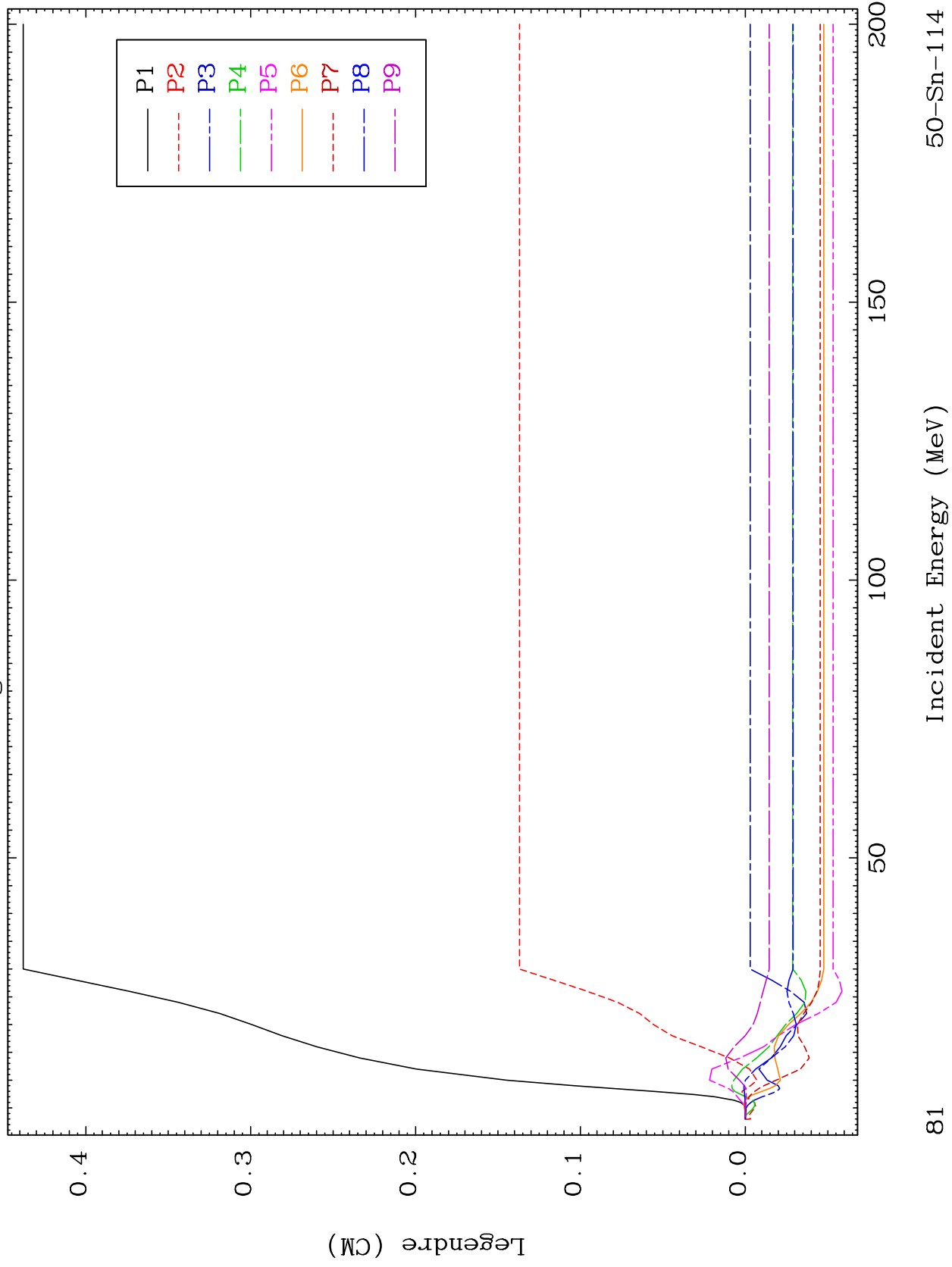
MAT 5031

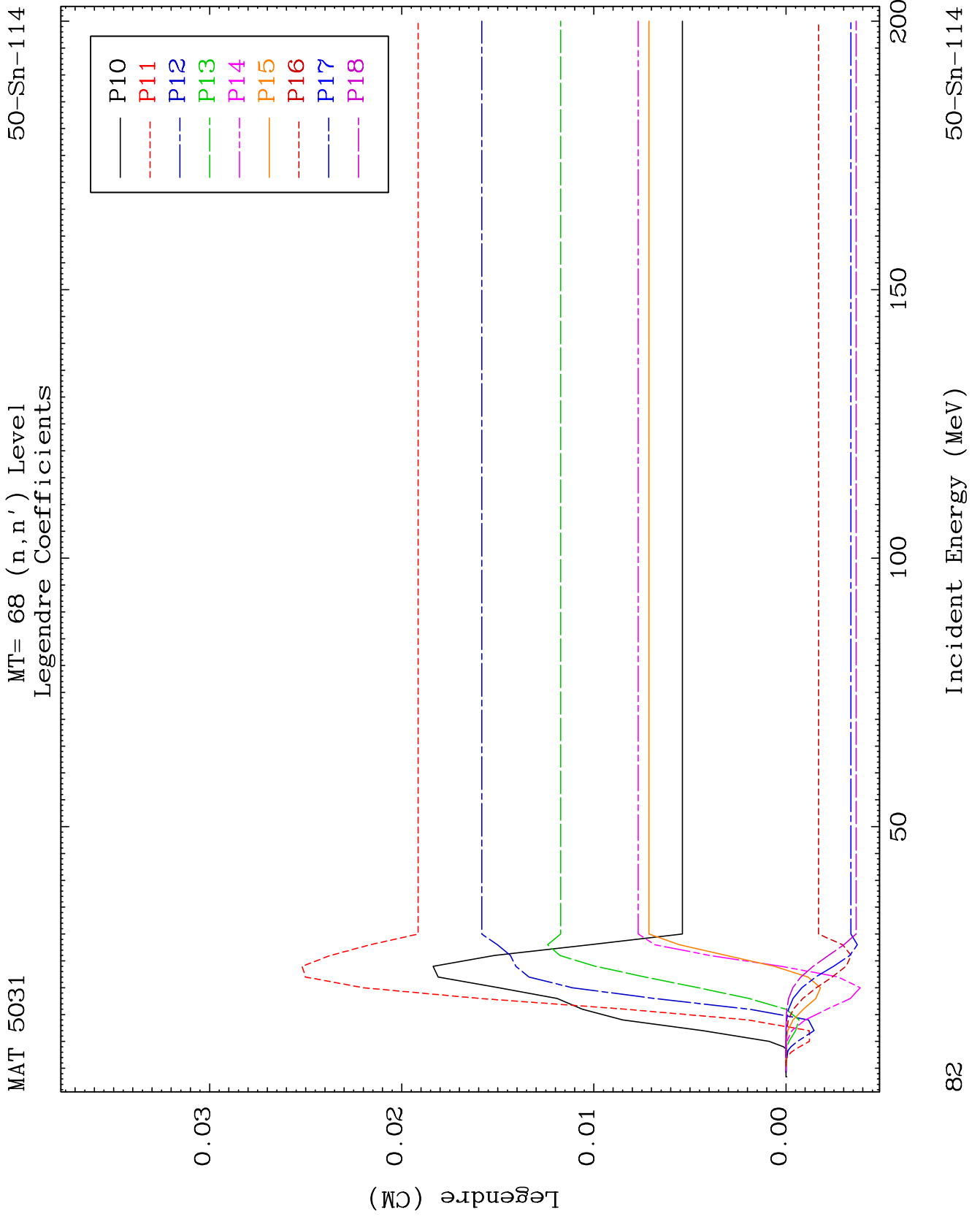
MT= 67 (n,n') Level
Legendre Coefficients

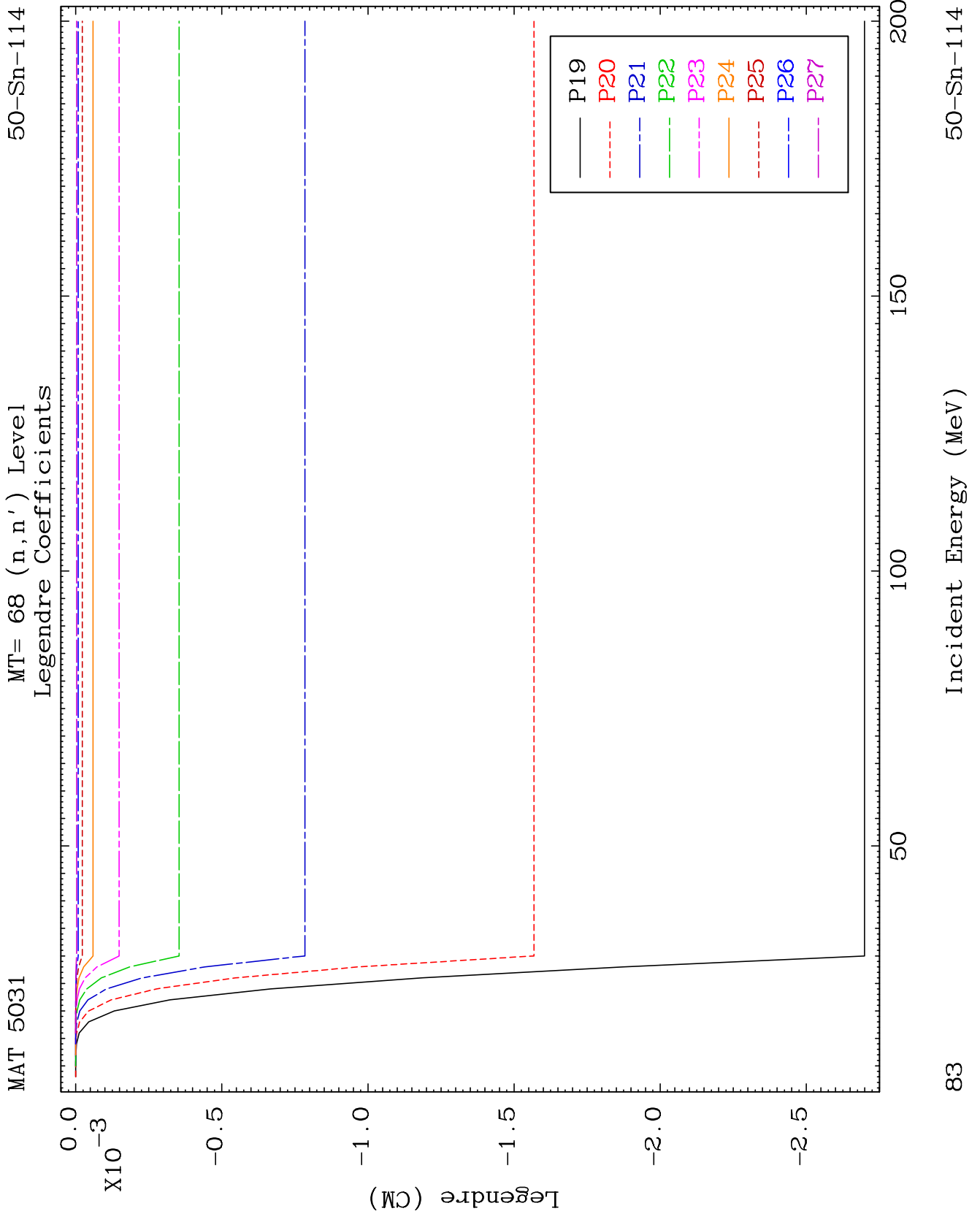
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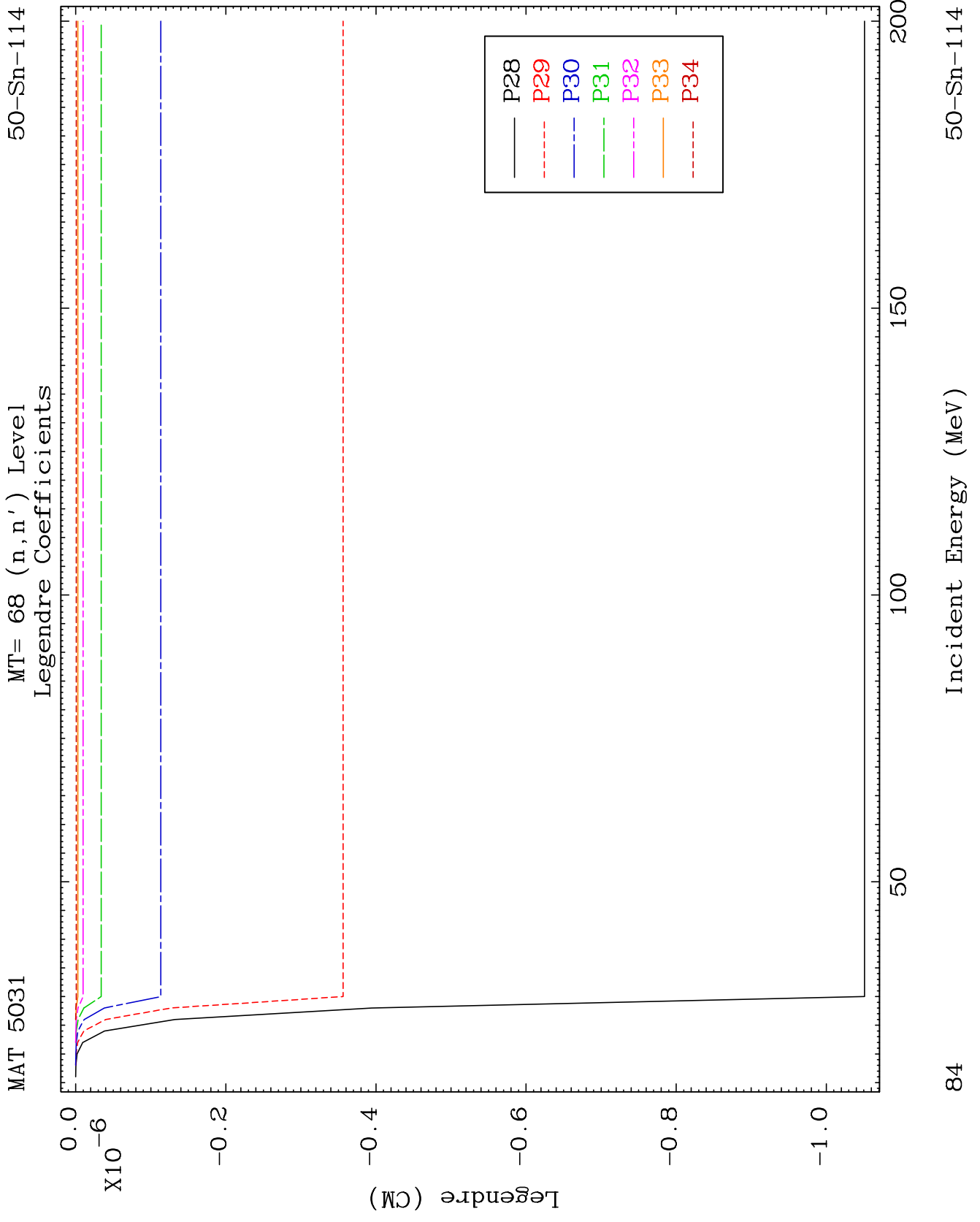








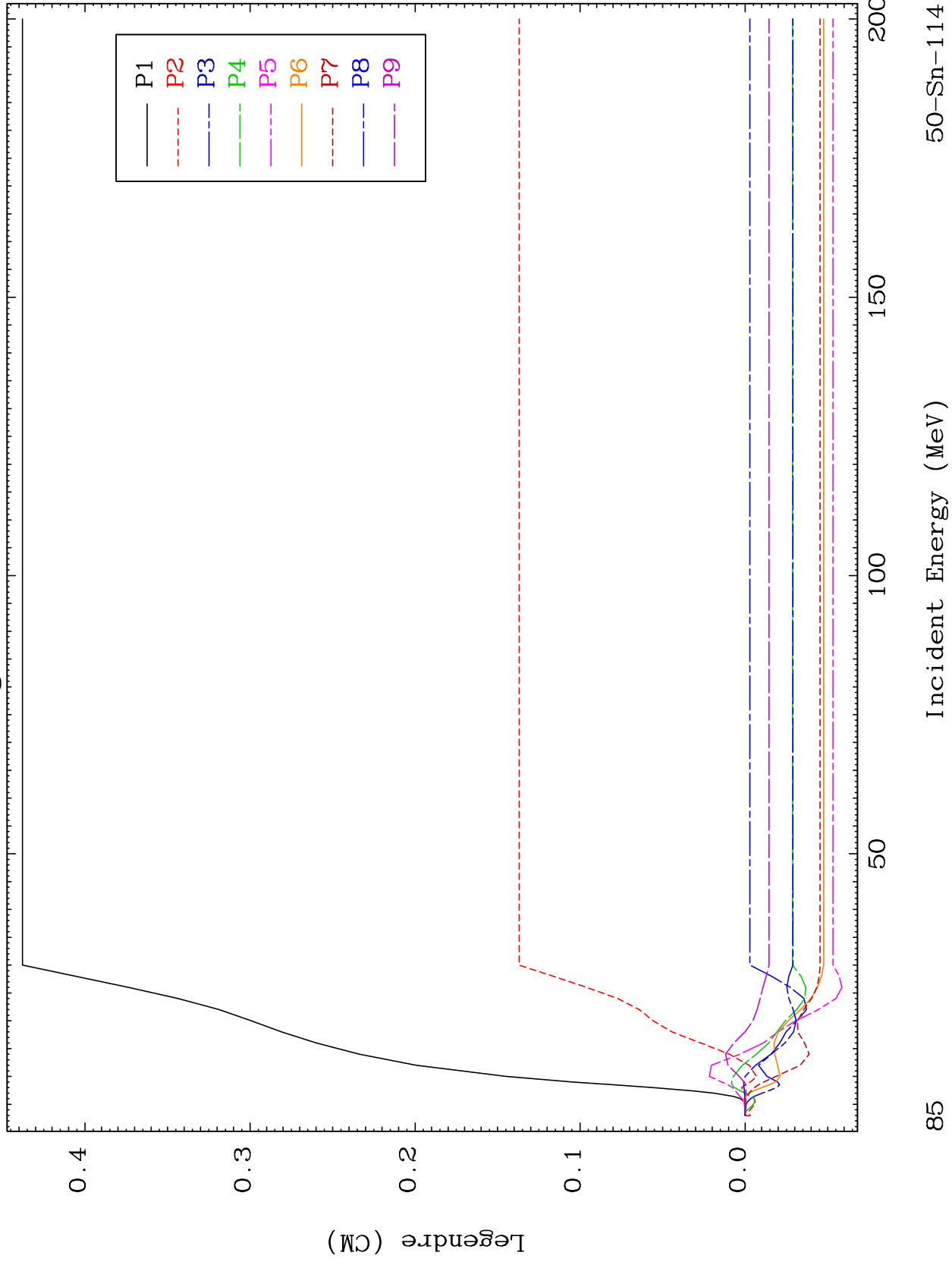




MAT 5031

MT= 69 (n,n') Level
Legendre Coefficients

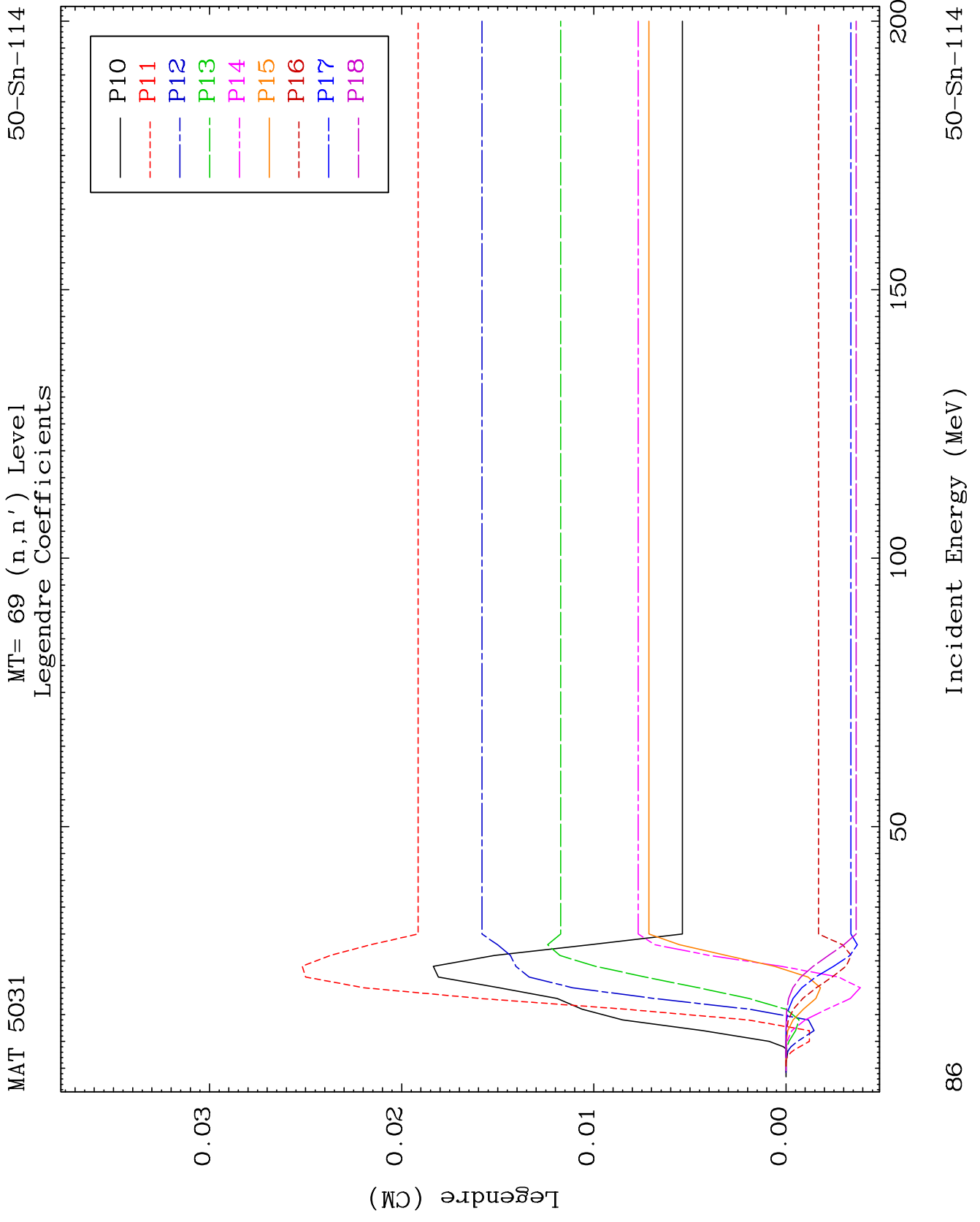
50-Sn-114

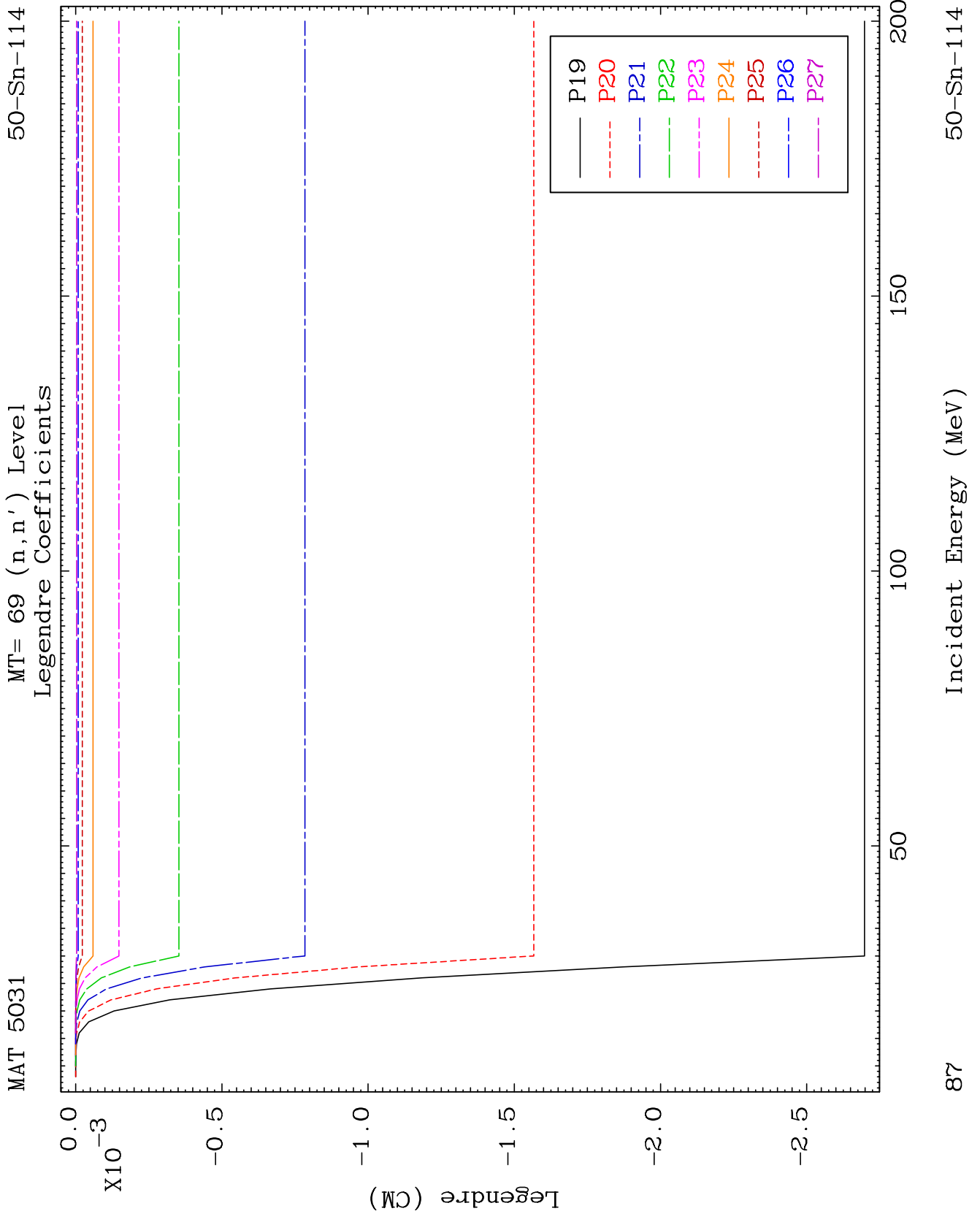


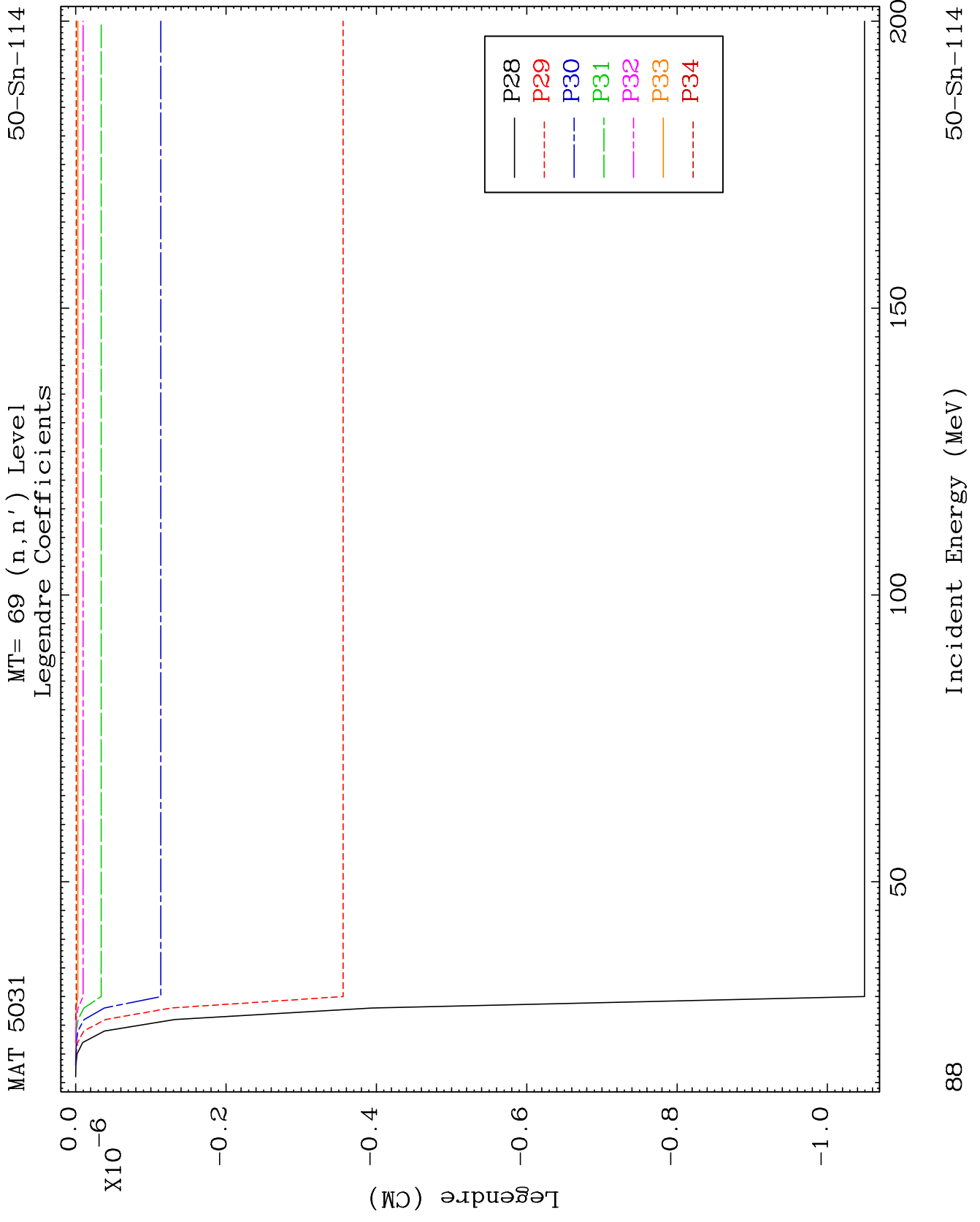
85

Incident Energy (MeV)

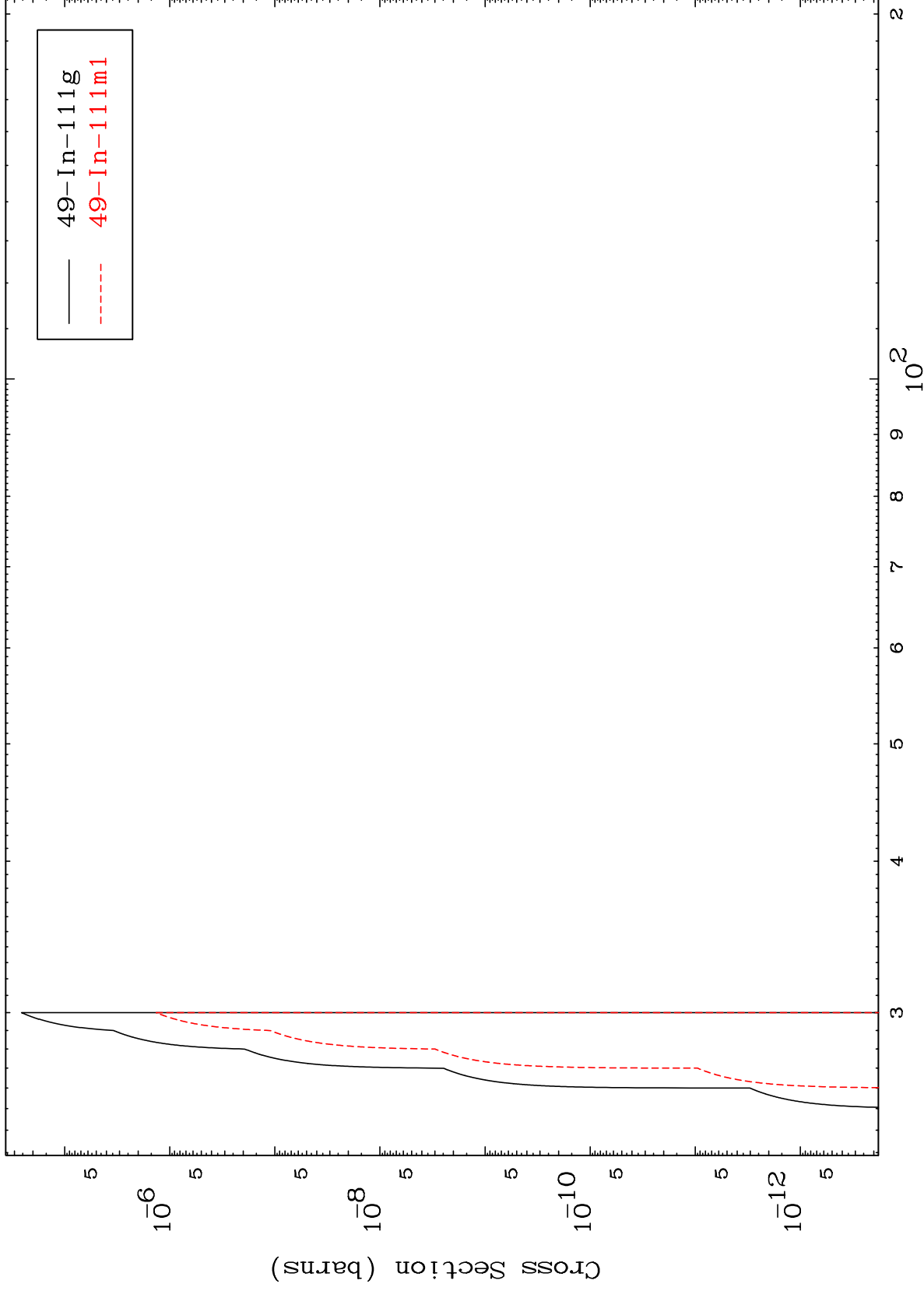
50-Sn-114







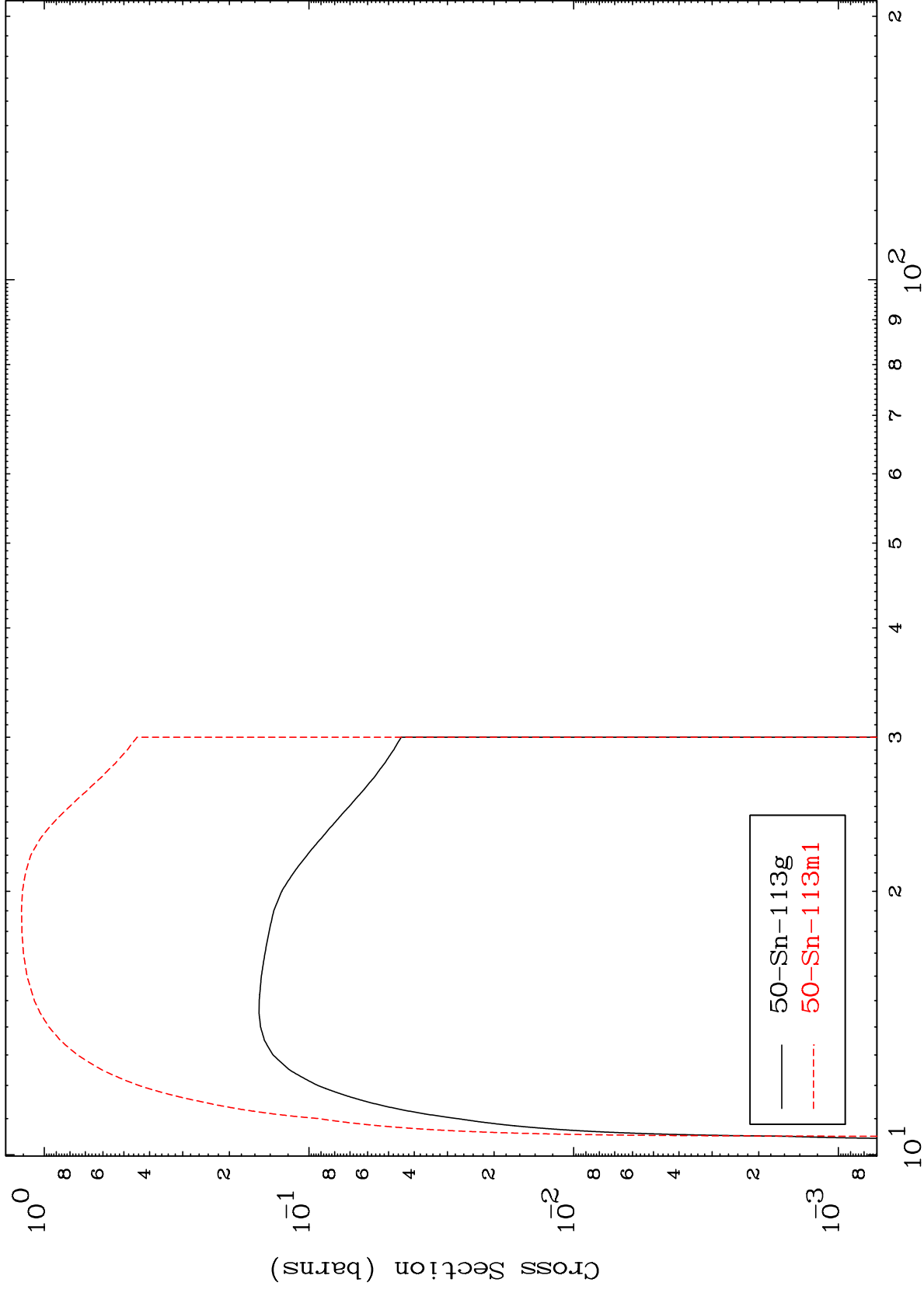
Radionuclide Production Cross Section



MAT 5031

50-Sn-114

(n,2n)
Radionuclide Production Cross Section



50-Sn-114

Incident Energy (MeV)

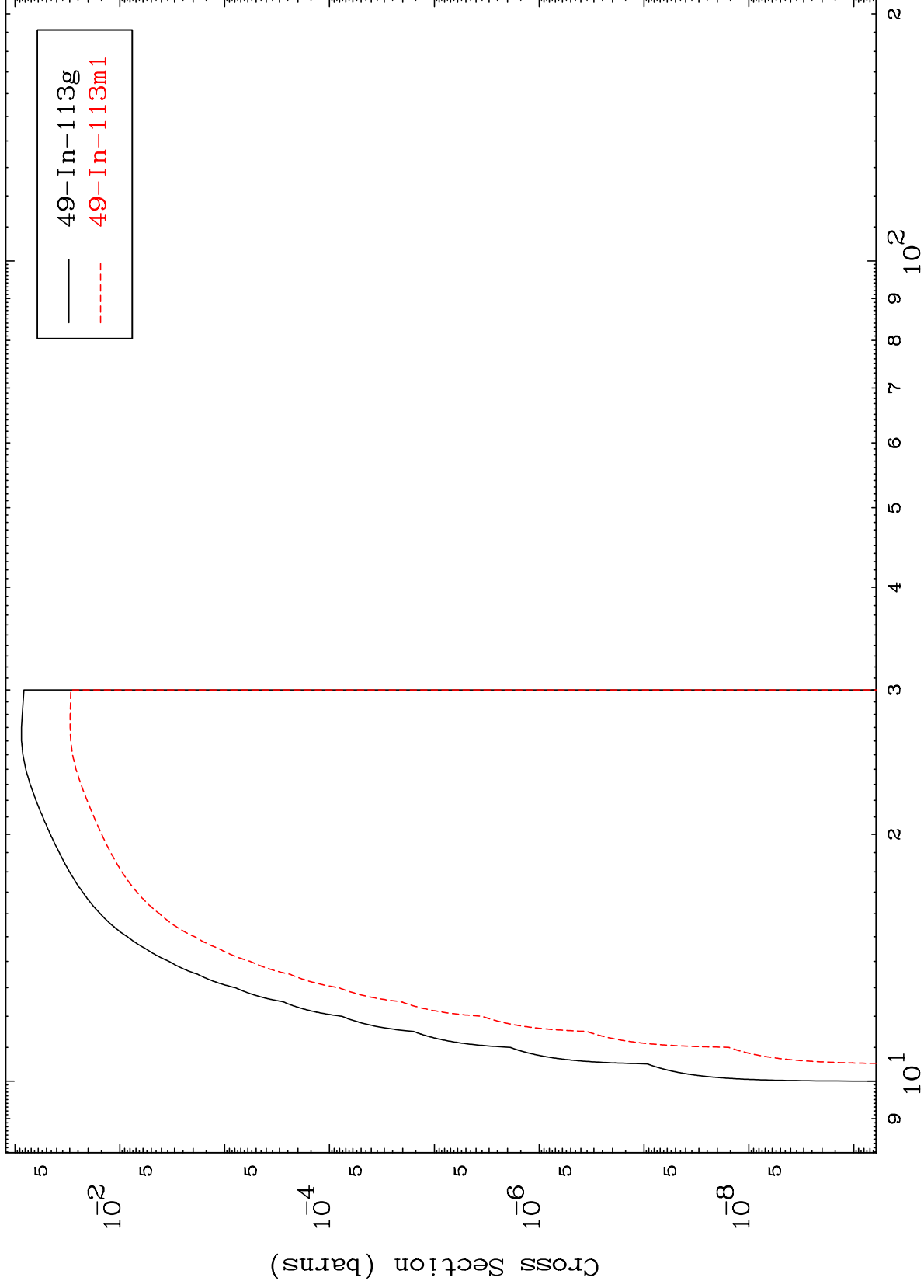
90

MAT 5031

(n,n') p

50-Sn-114

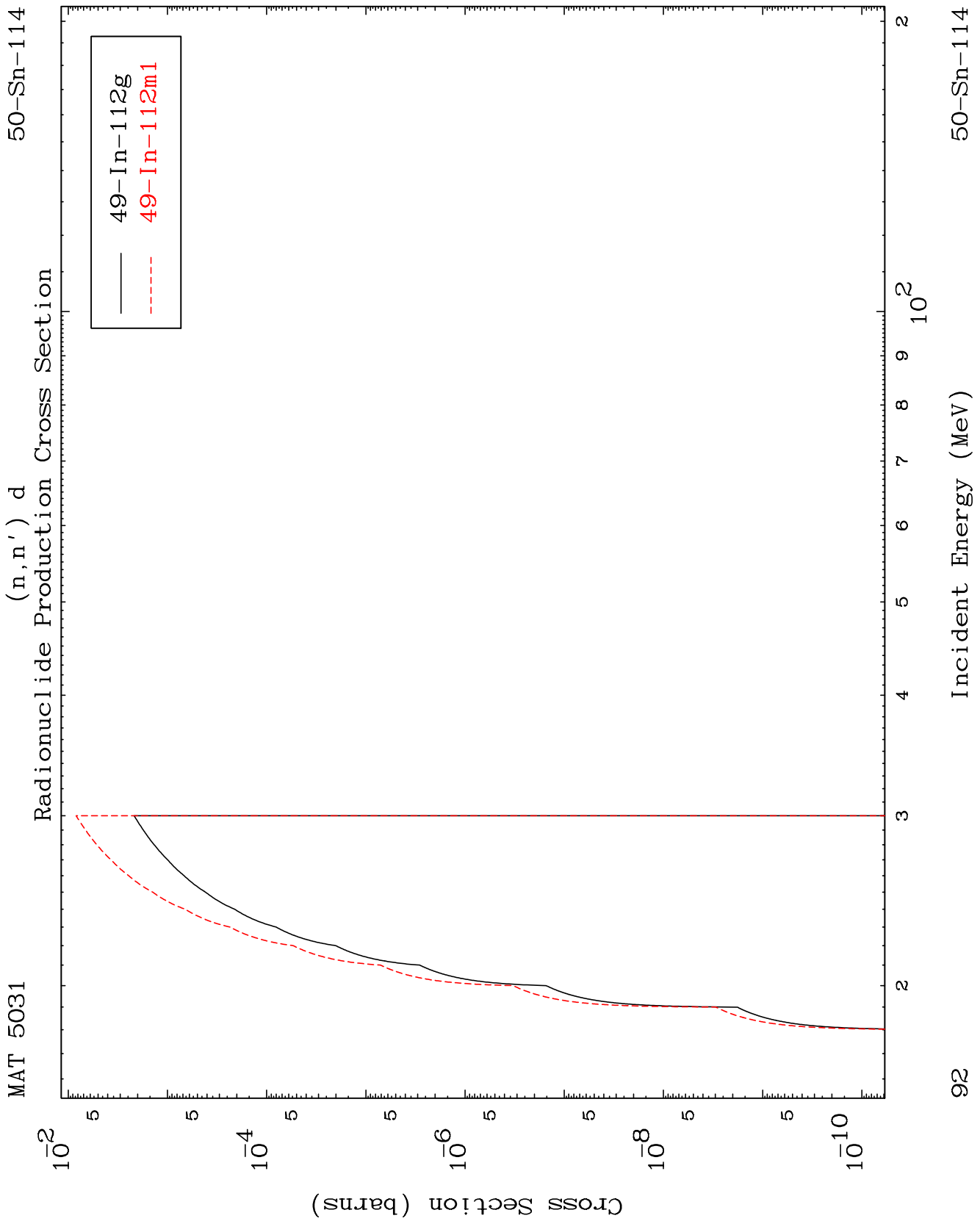
Radionuclide Production Cross Section



91

Incident Energy (MeV)

50-Sn-114

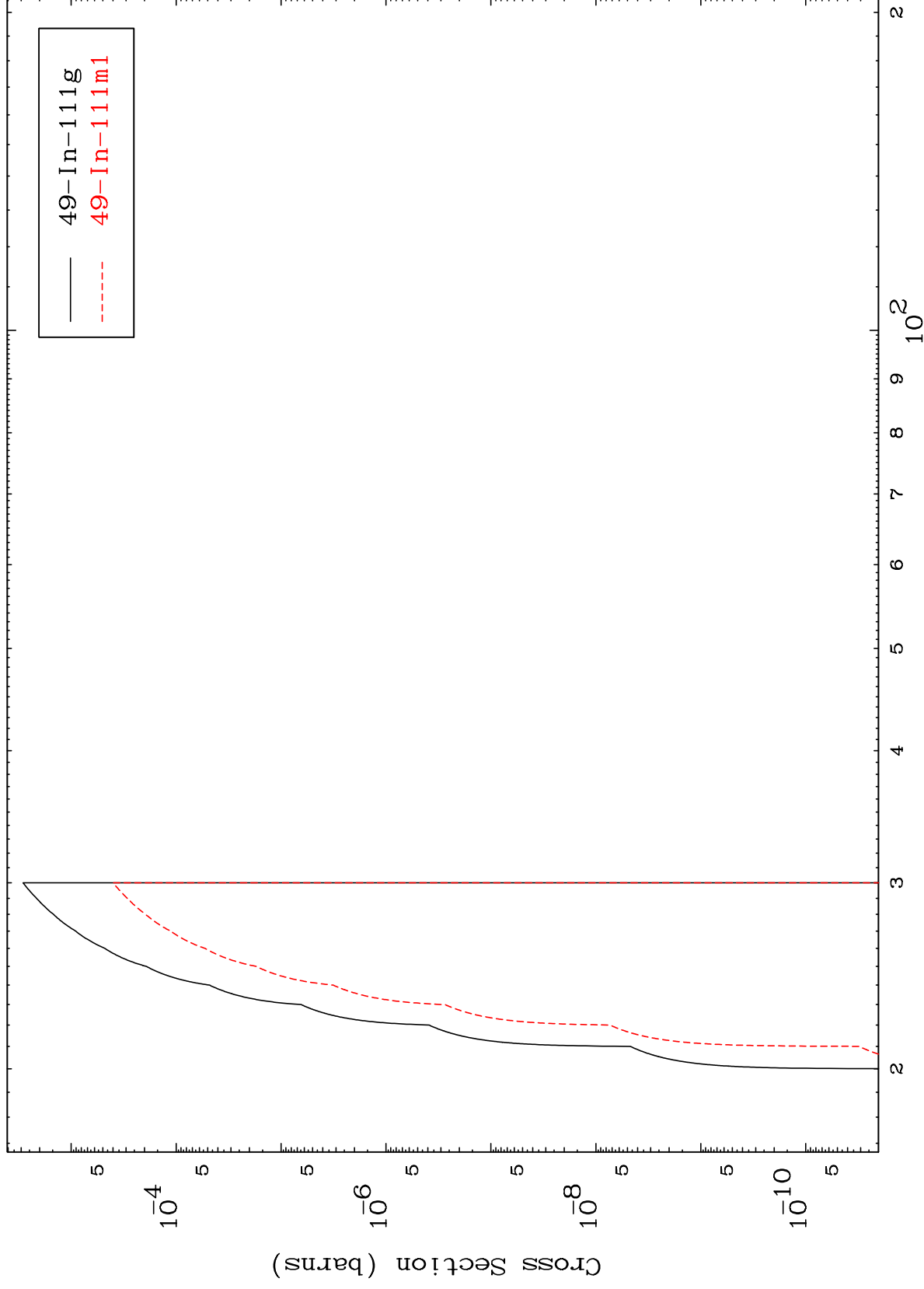


MAT 5031

(n,n') t

50-Sn-114

Radionuclide Production Cross Section



93

Incident Energy (MeV)

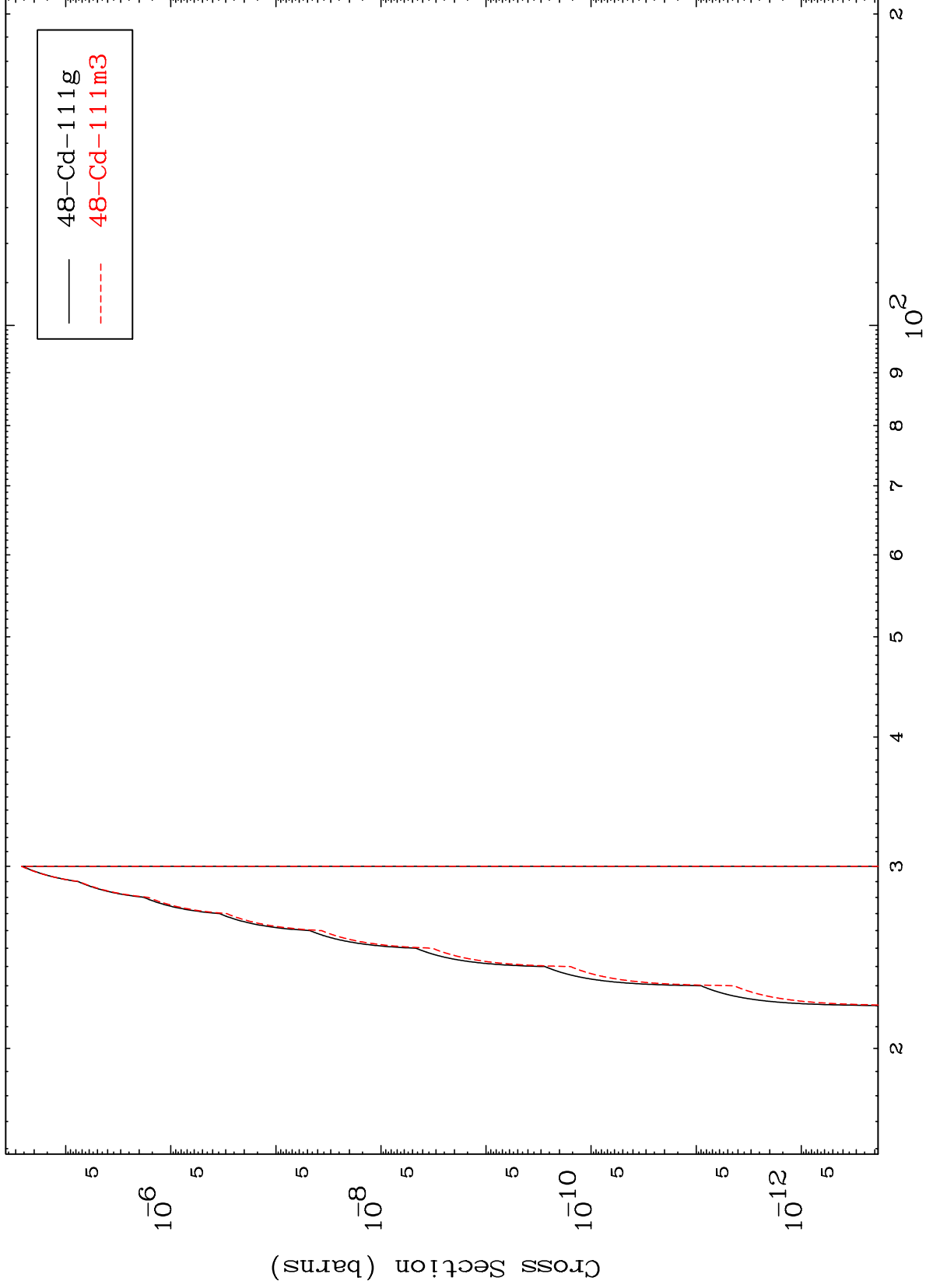
50-Sn-114

MAT 5031

(n,n') He-3

50-Sn-114

Radionuclide Production Cross Section



94

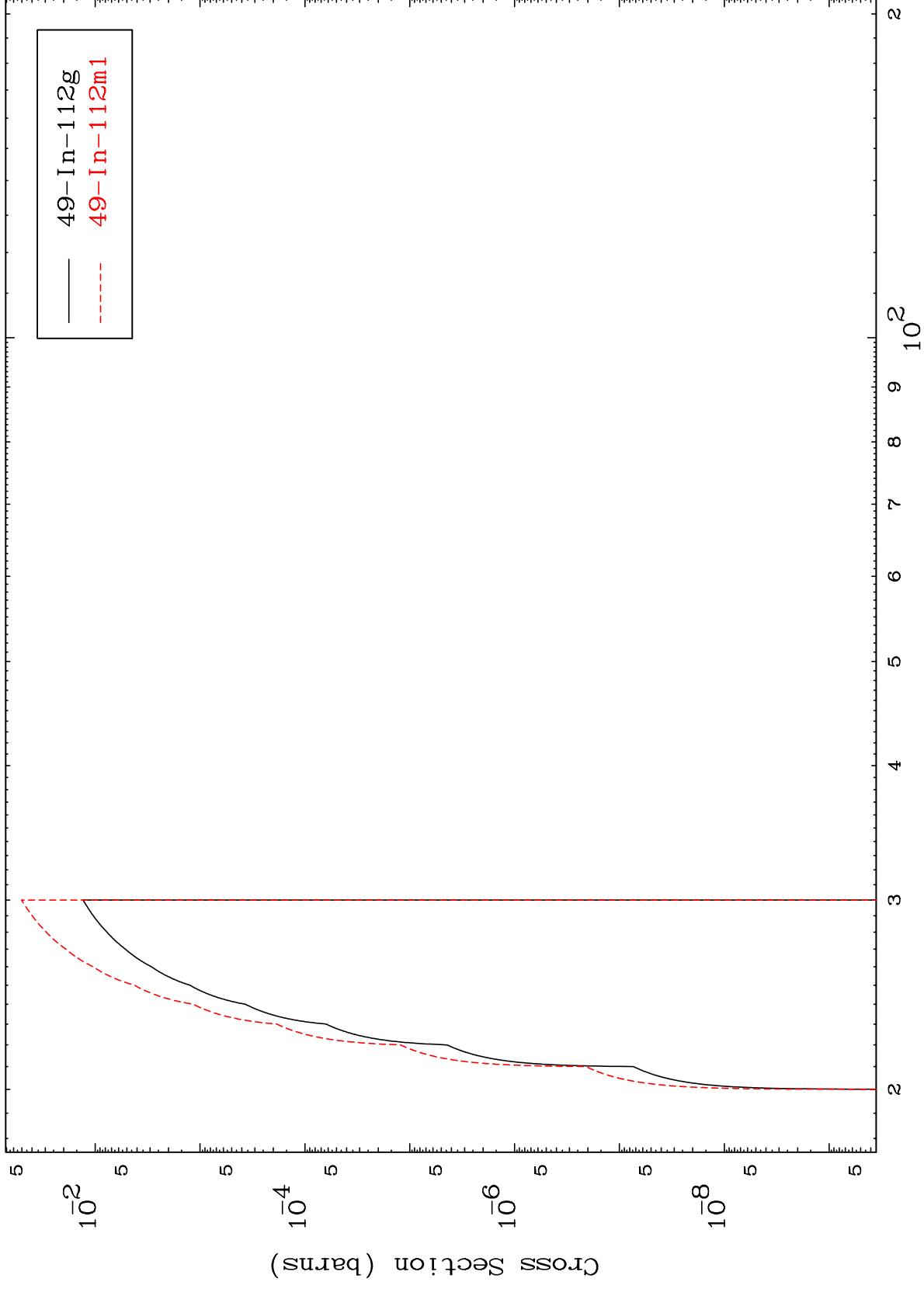
Incident Energy (MeV)

50-Sn-114

MAT 5031

50-Sn-114

(n,2n) p
Radionuclide Production Cross Section

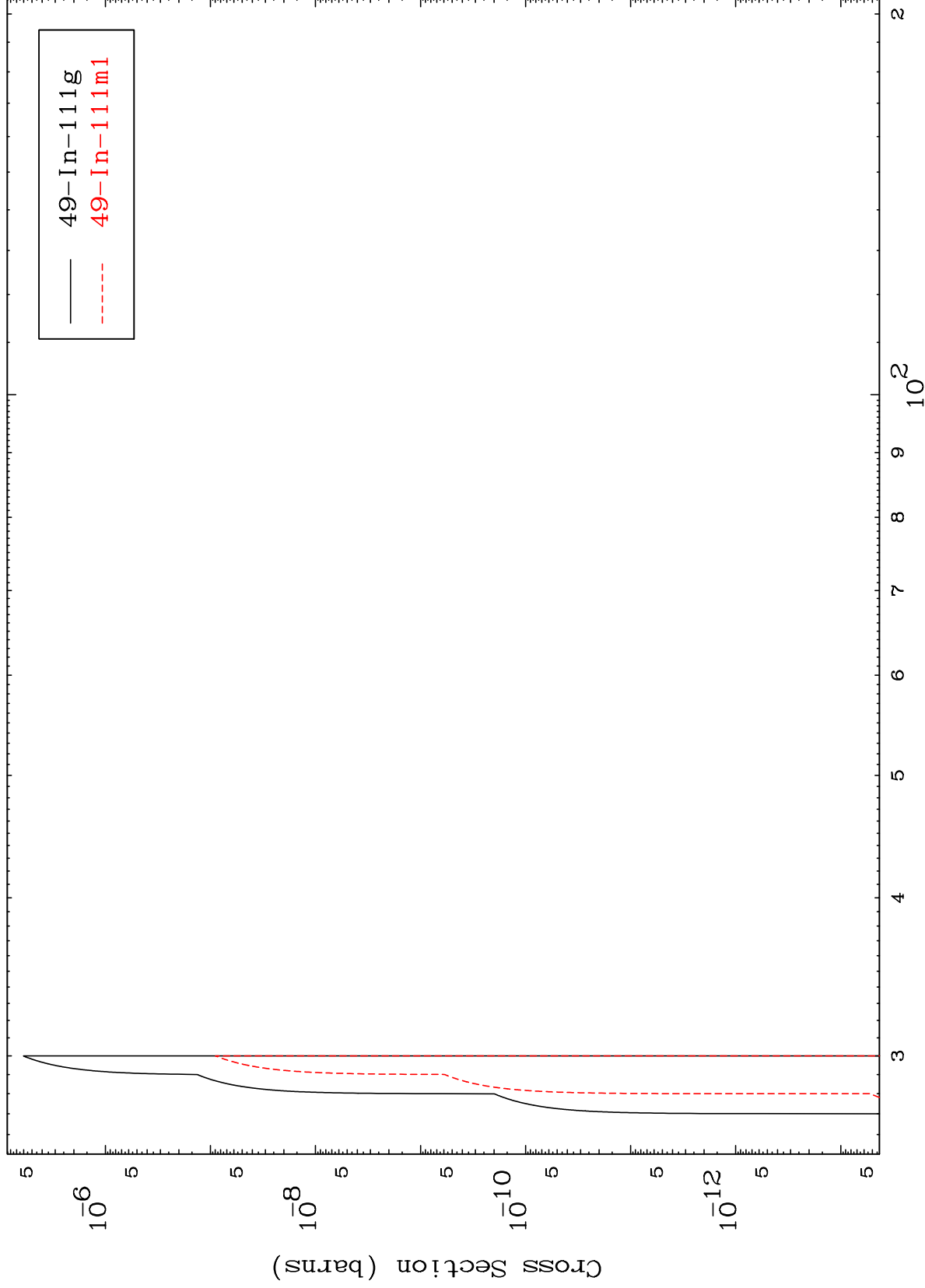


95

Incident Energy (MeV)

50-Sn-114

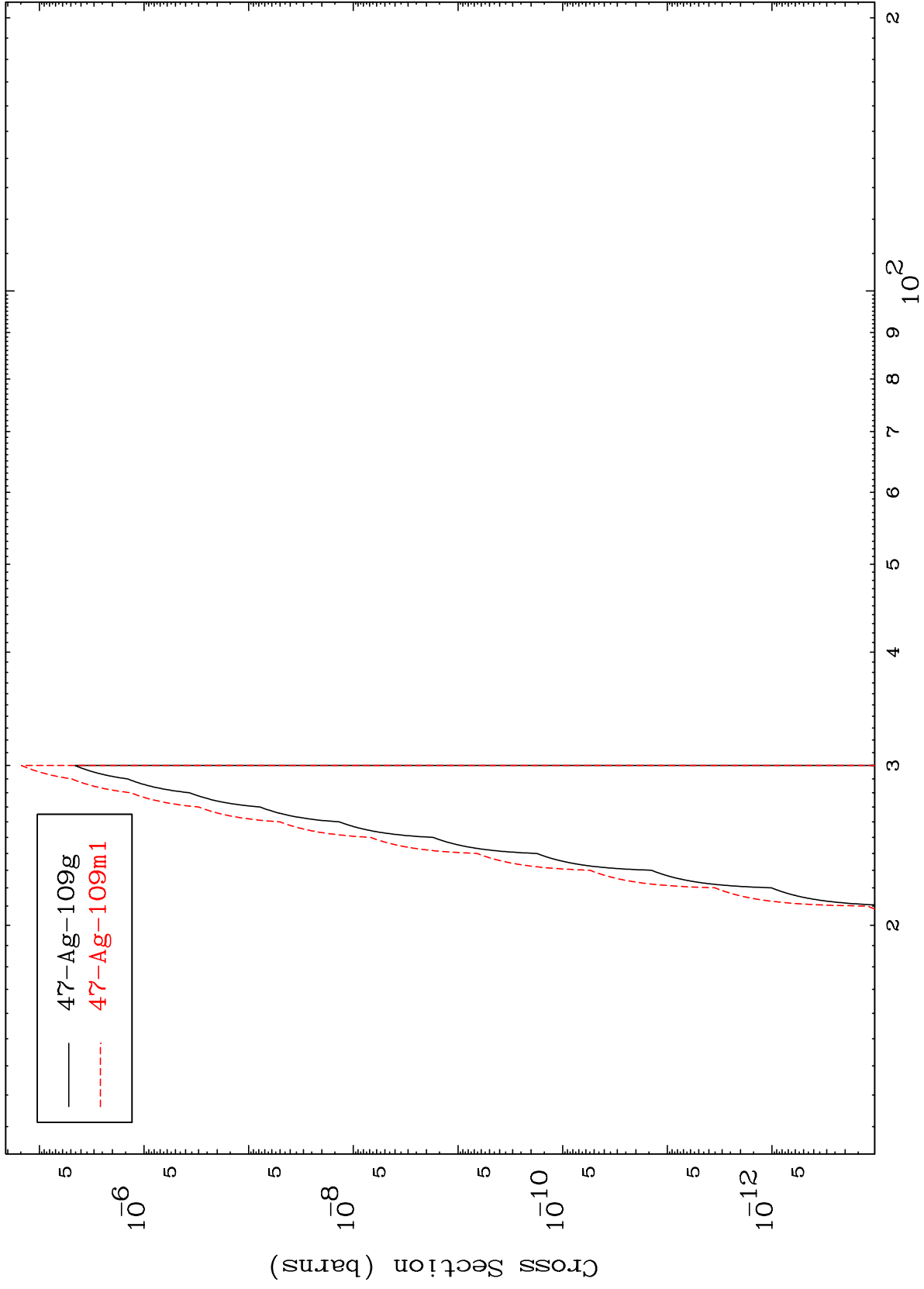
Radionuclide Production Cross Section



MAT 5031

50-Sn-114

(n,n') p α
Radionuclide Production Cross Section



97

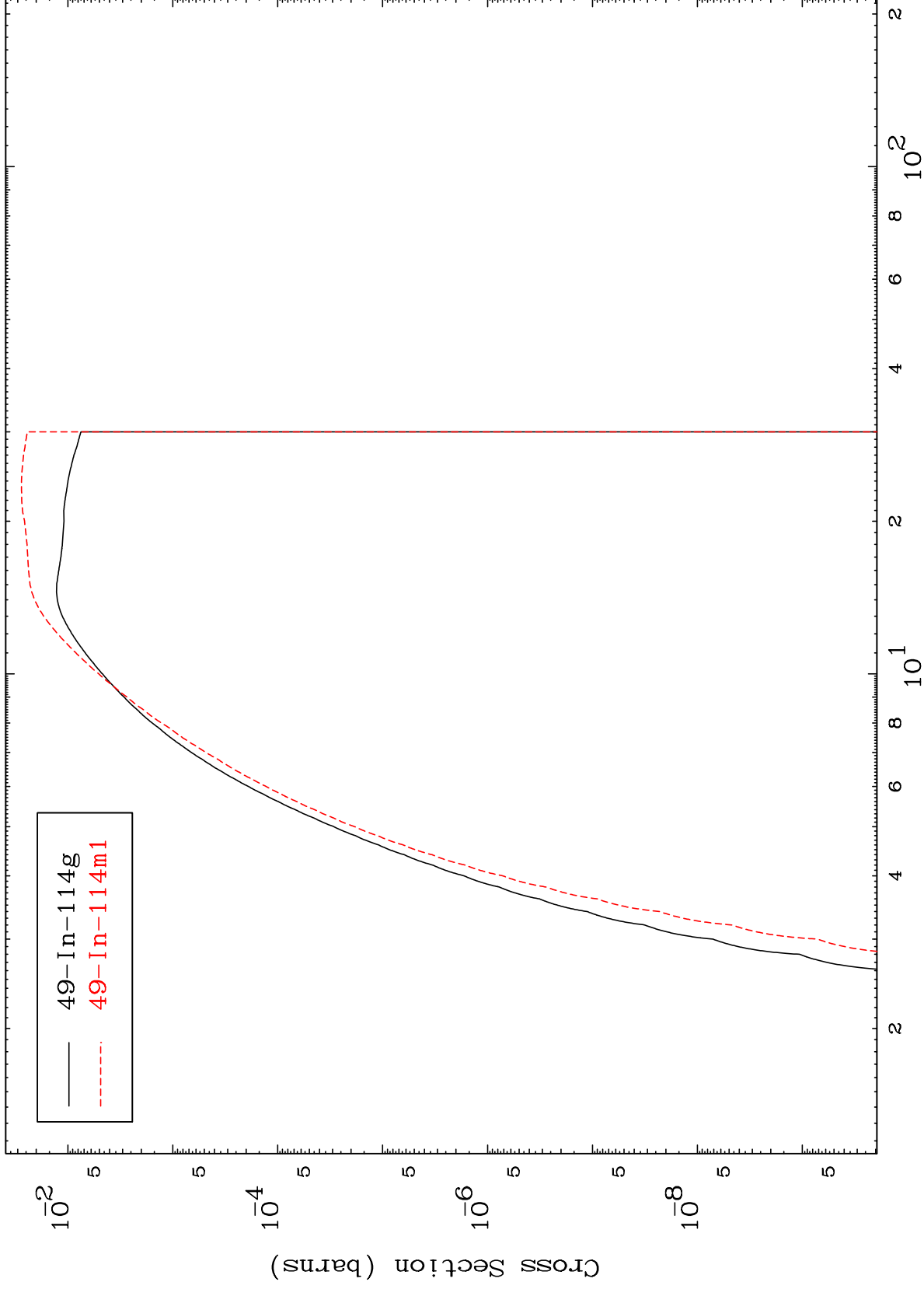
Incident Energy (MeV)

50-Sn-114

MAT 5031

50-Sn-114

(n,p)
Radionuclide Production Cross Section



98

50-Sn-114

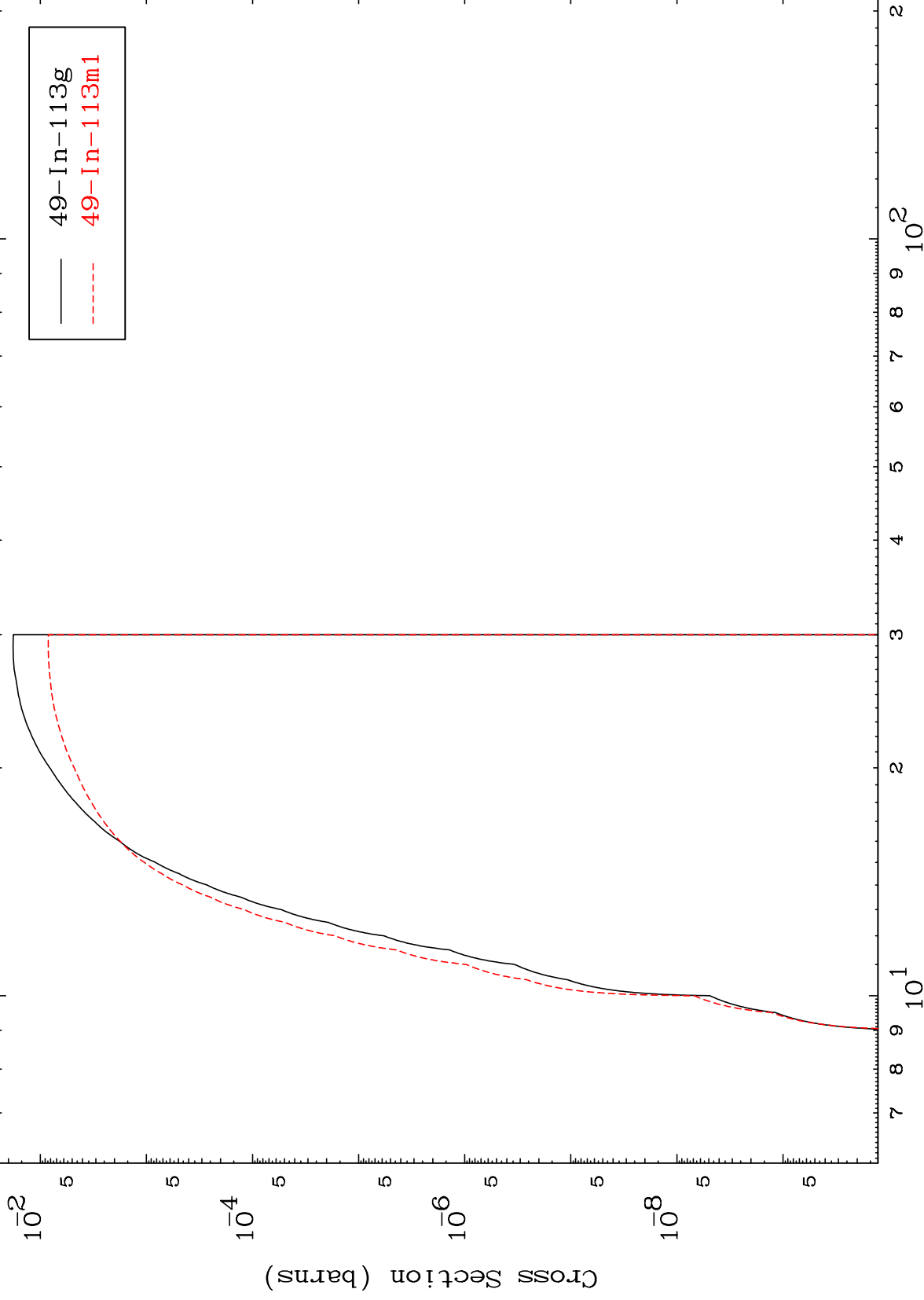
Incident Energy (MeV)

MAT 5031

(n,d)

50-Sn-114

Radionuclide Production Cross Section



99

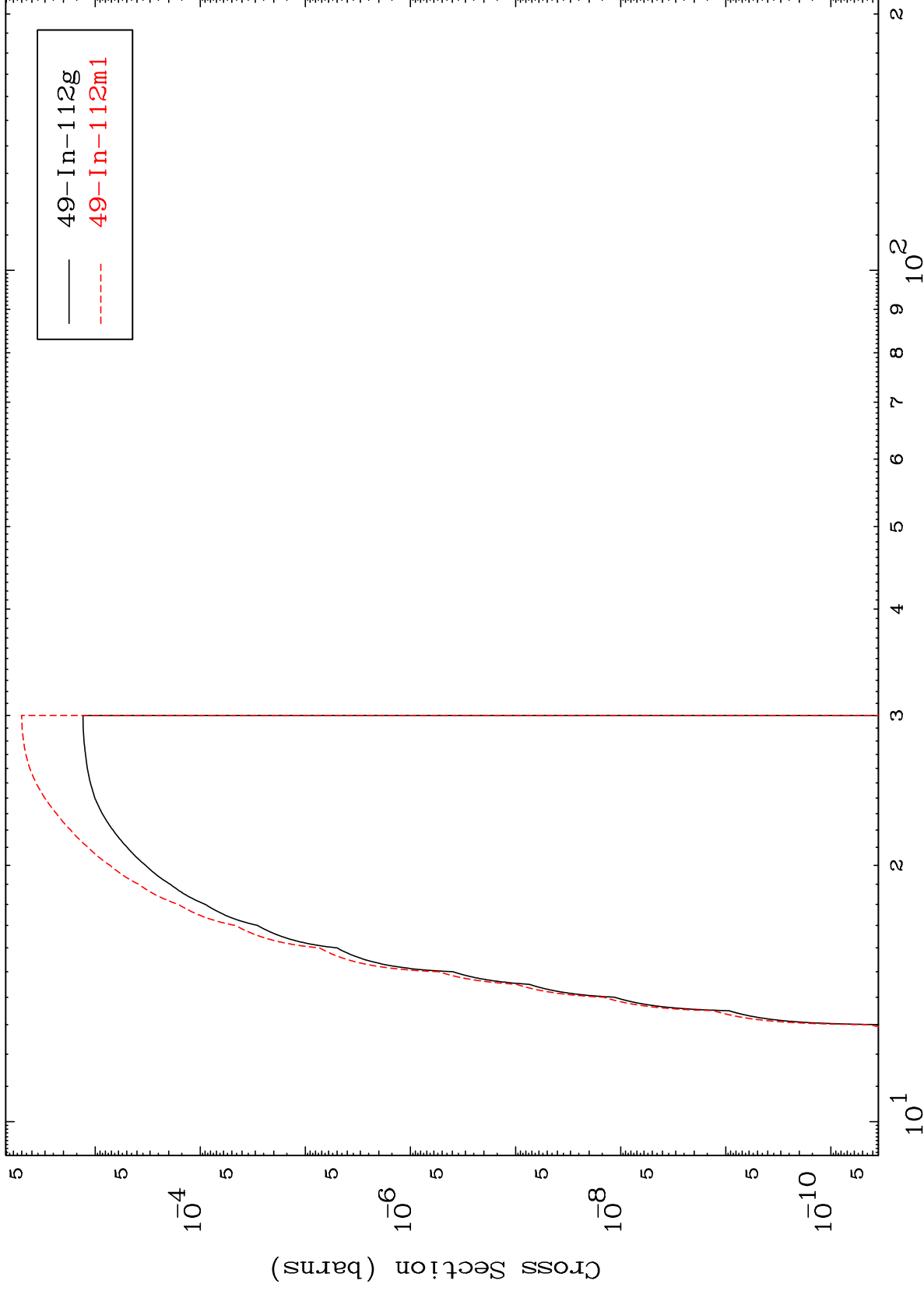
Incident Energy (MeV)

50-Sn-114

MAT 5031

50-Sn-114

(n,t)
Radionuclide Production Cross Section



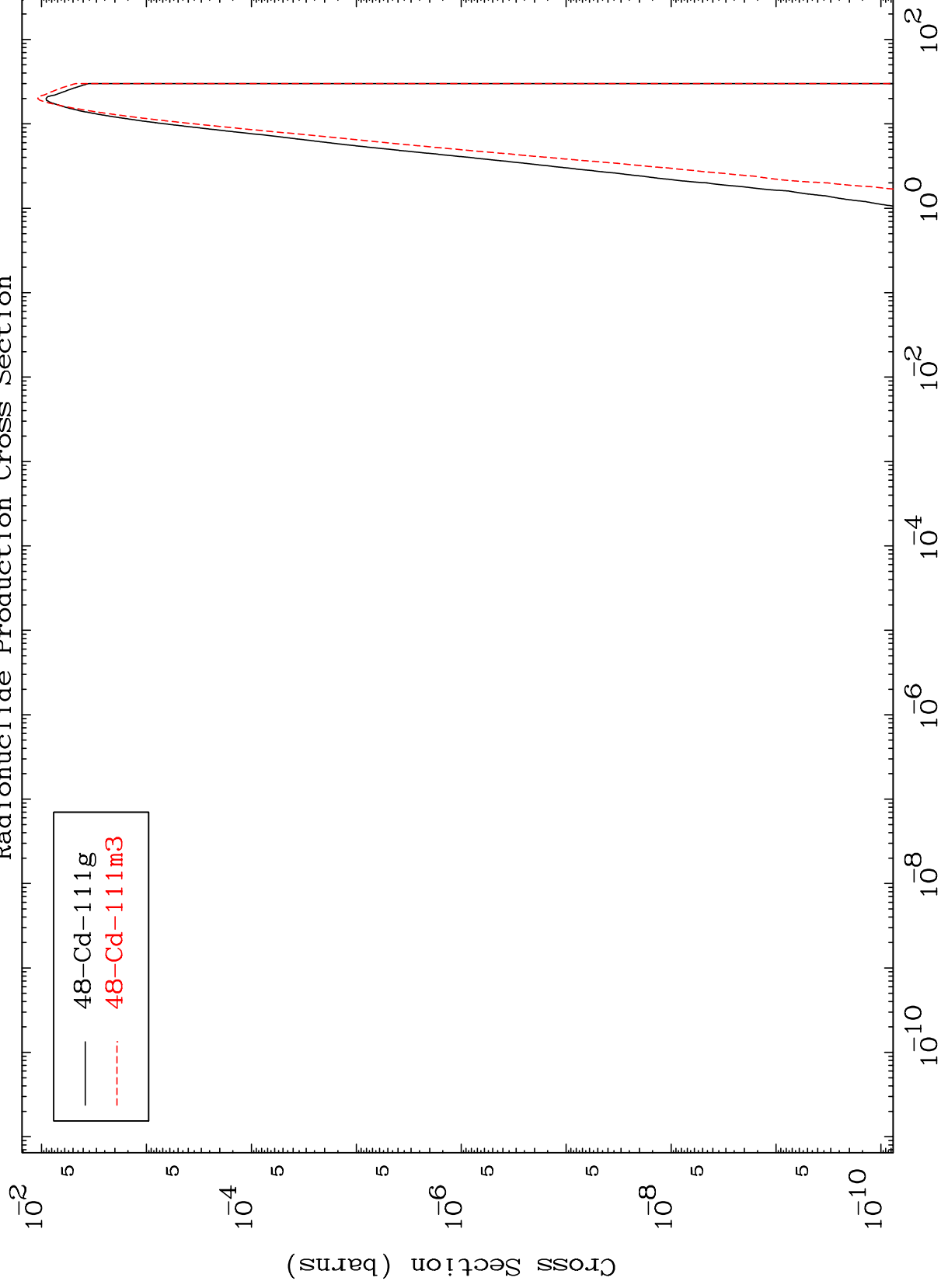
50-Sn-114

Incident Energy (MeV)

MAT 5031

50-Sn-114

Radionuclide Production Cross Section
(n, α)



101

Incident Energy (MeV)

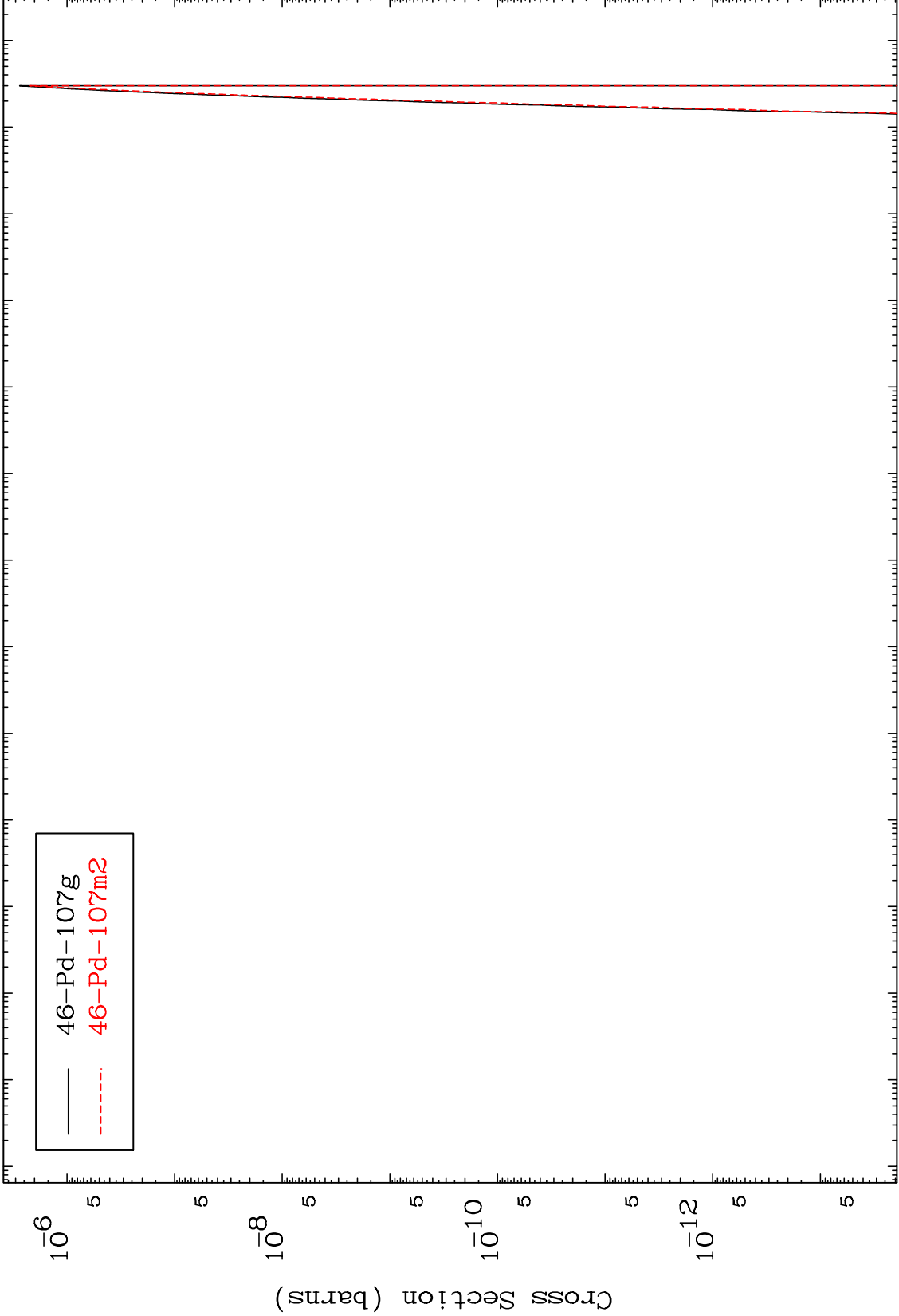
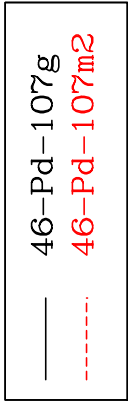
50-Sn-114

MAT 5031

(n,2α)

50-Sn-114

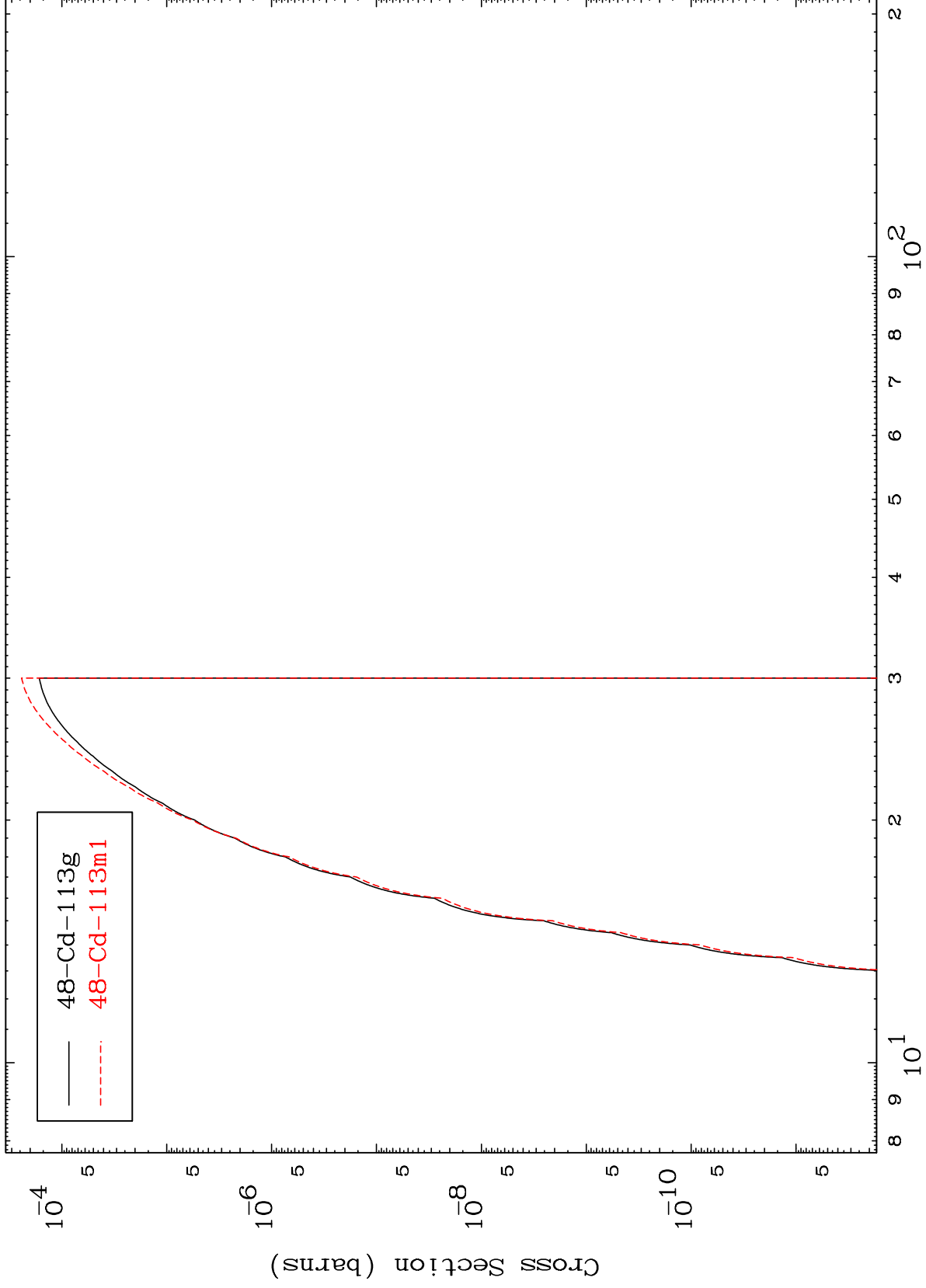
Radionuclide Production Cross Section



MAT 5031

50-Sn-114

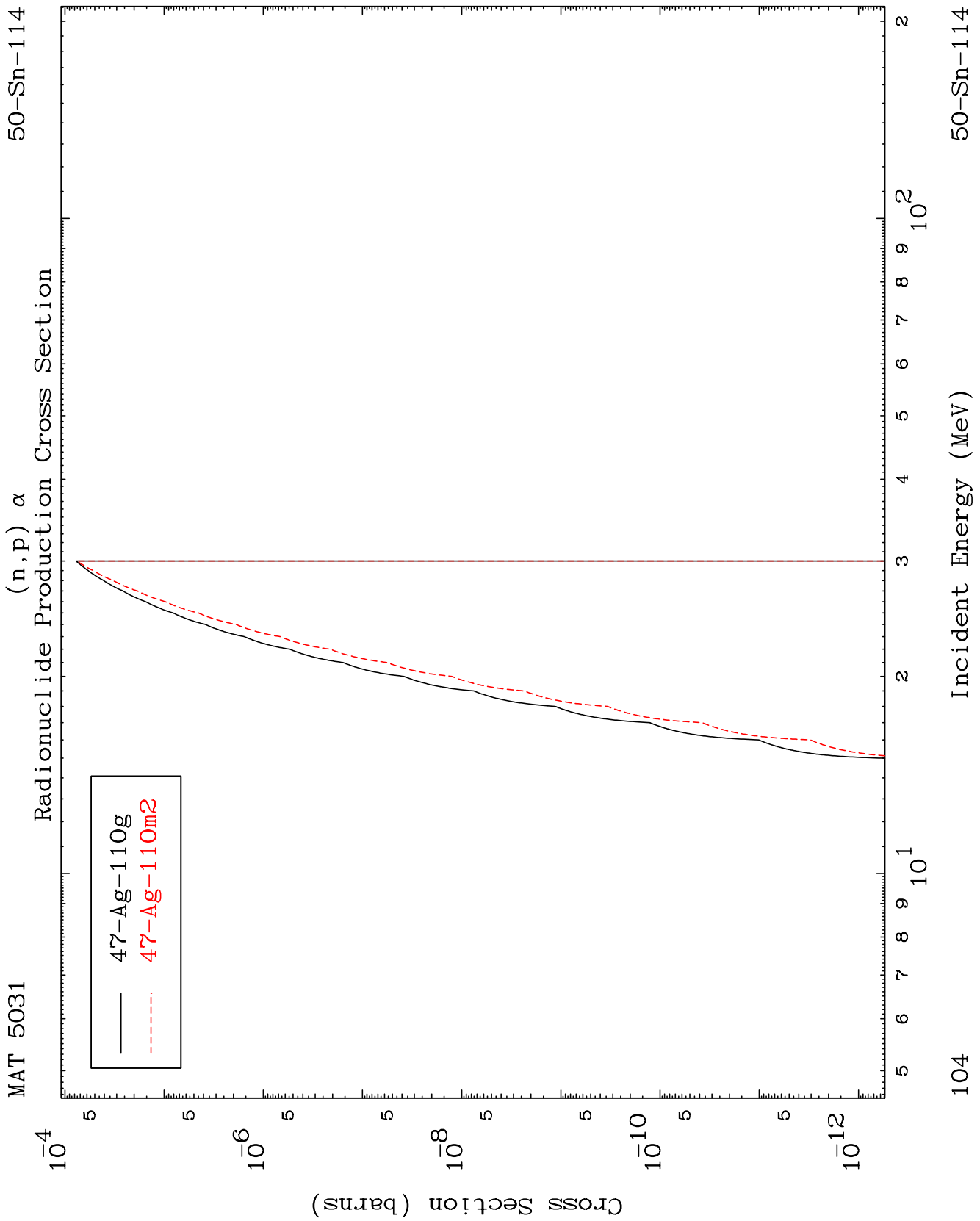
(n,2p)
Radionuclide Production Cross Section



103

Incident Energy (MeV)

50-Sn-114

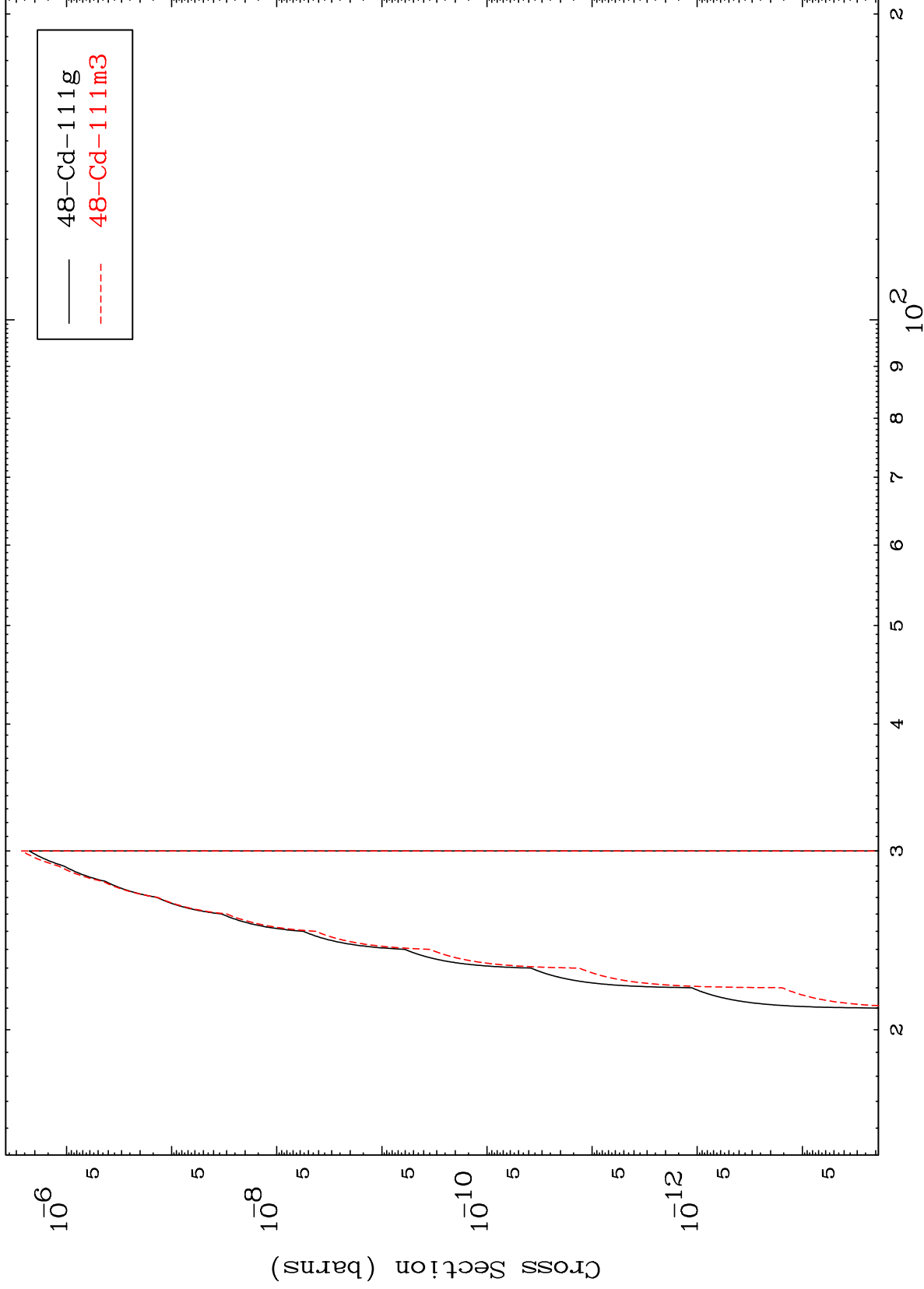


MAT 5031

(n,p) t

50-Sn-114

Radionuclide Production Cross Section



105

Incident Energy (MeV)

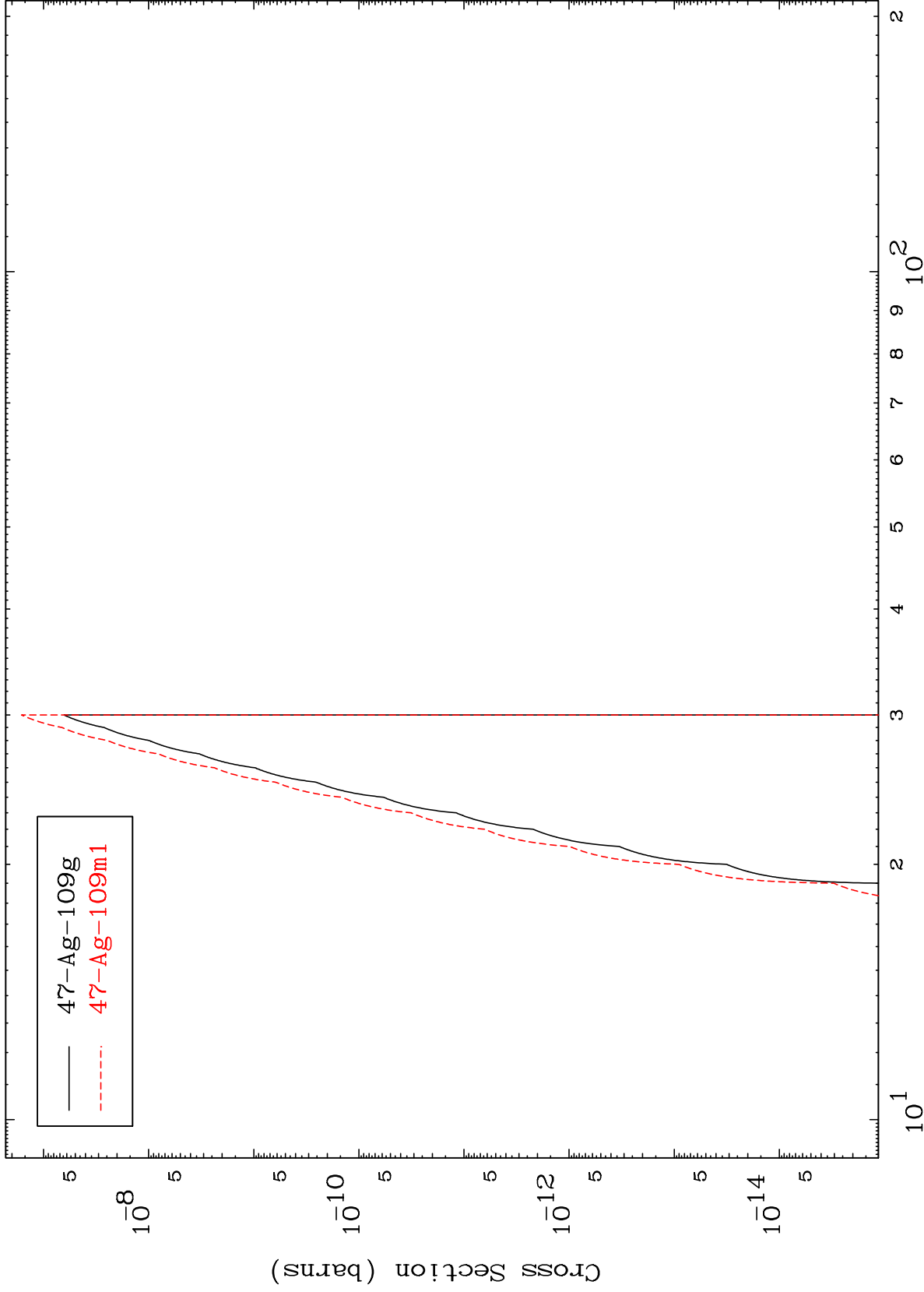
50-Sn-114

MAT 5031

(n,d) α

50-Sn-114

Radionuclide Production Cross Section



Incident Energy (MeV)

50-Sn-114

106