

Program EVALPLOT  
(Version 2021-1)

by

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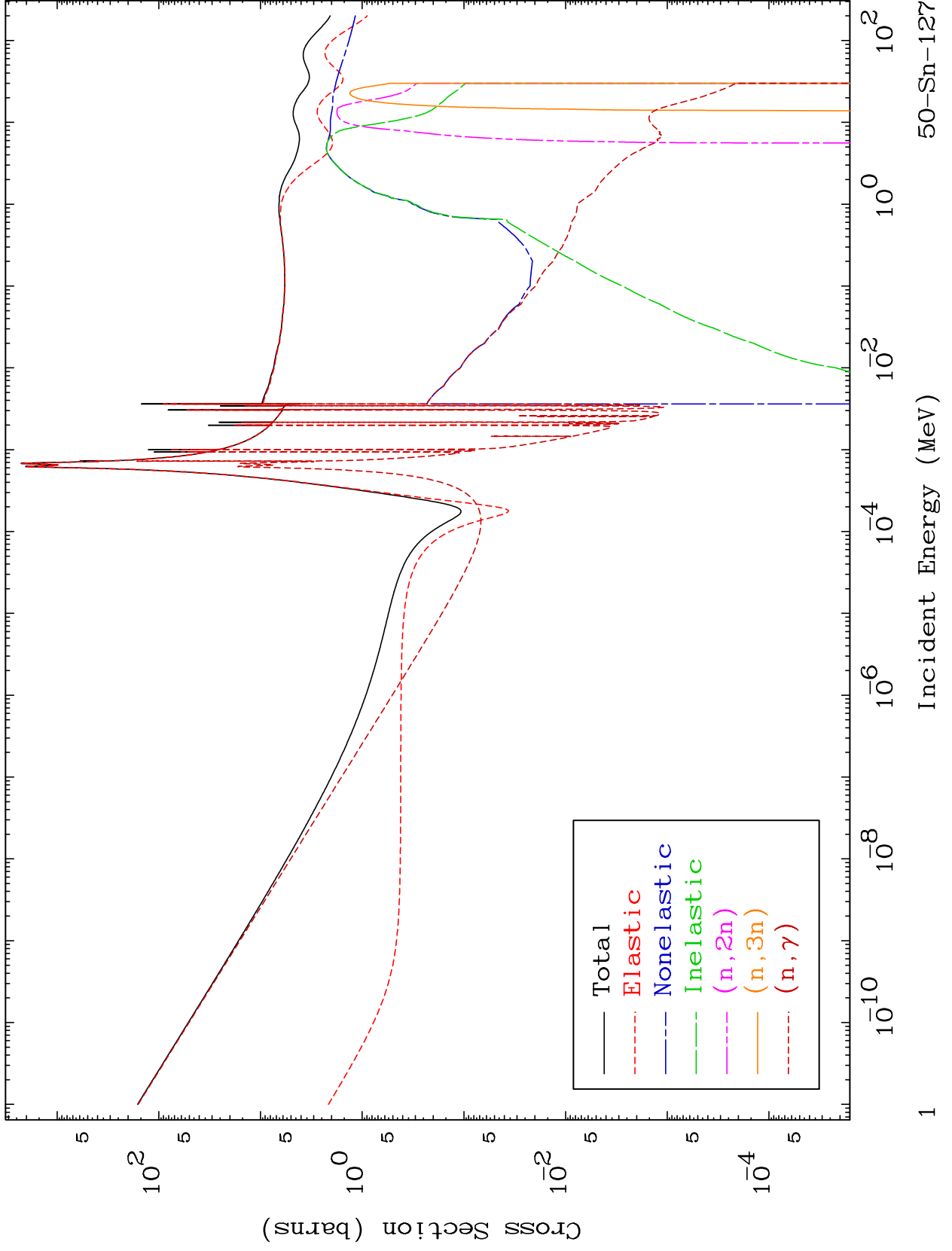
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 5070

Neutron Major  
293 Kelvin Cross Sections

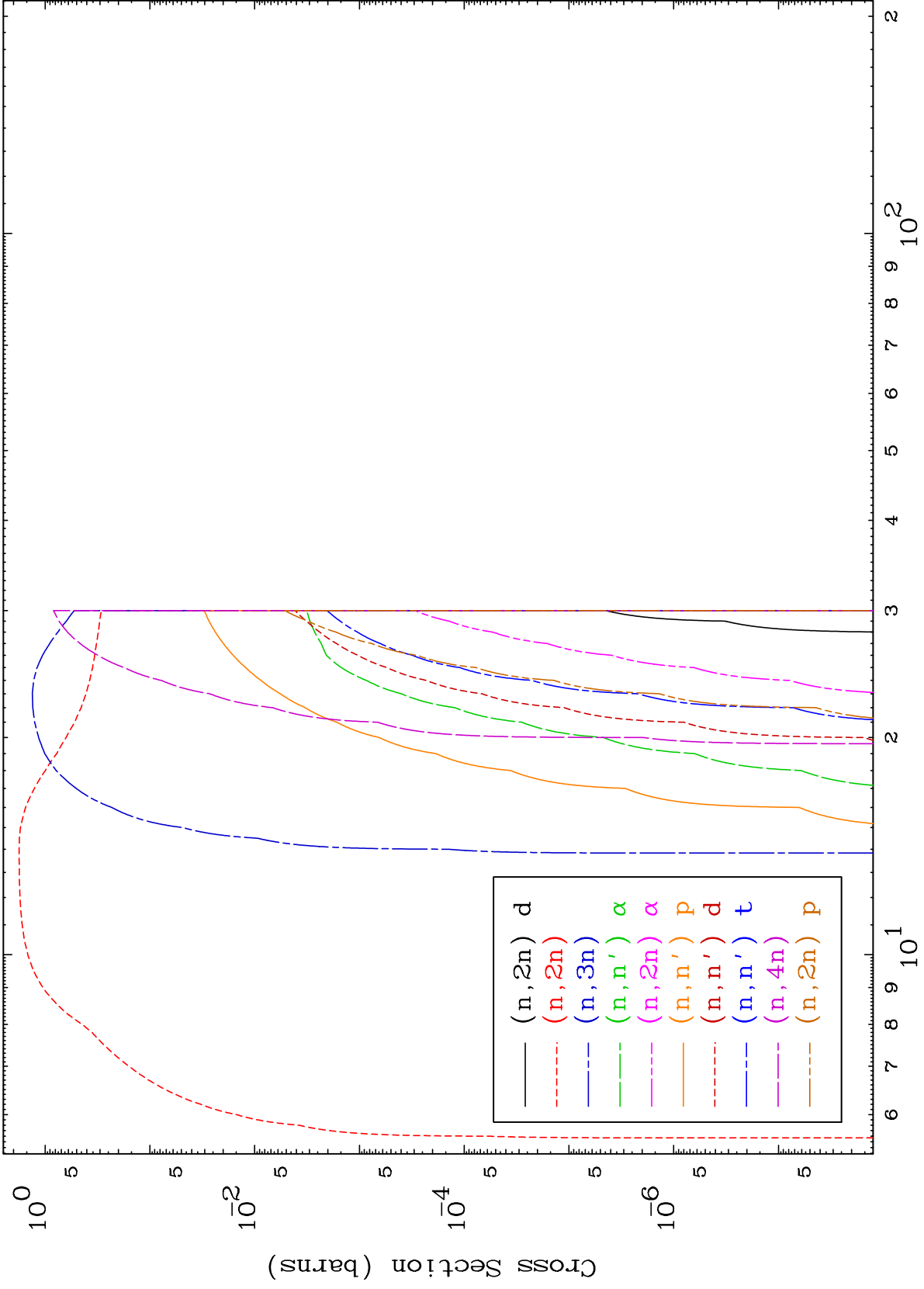
50-Sn-127



MAT 5070

Neutron Absorption  
293 Kelvin Cross Sections

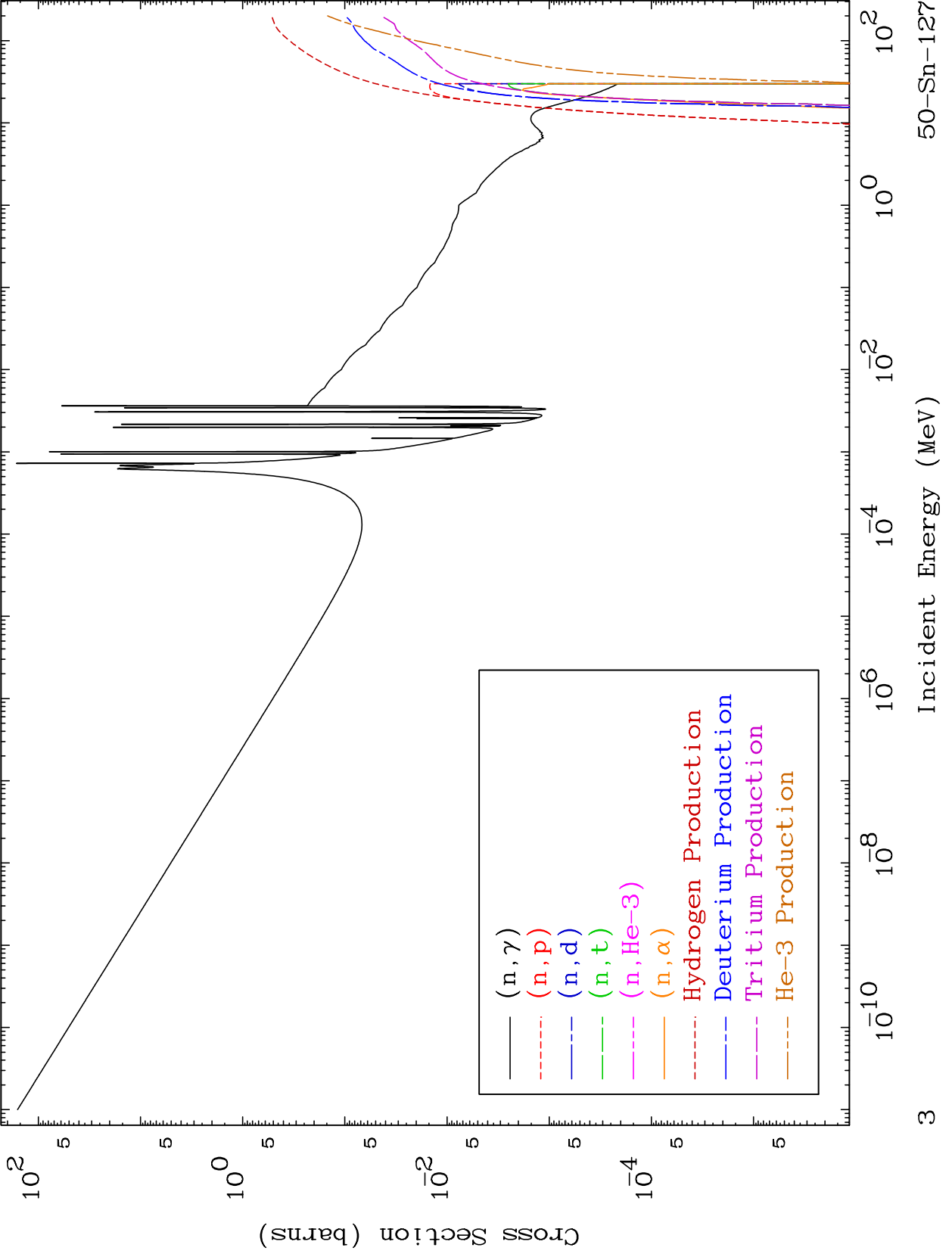
50-Sn-127



MAT 5070

Neutron Absorption  
293 Kelvin Cross Sections

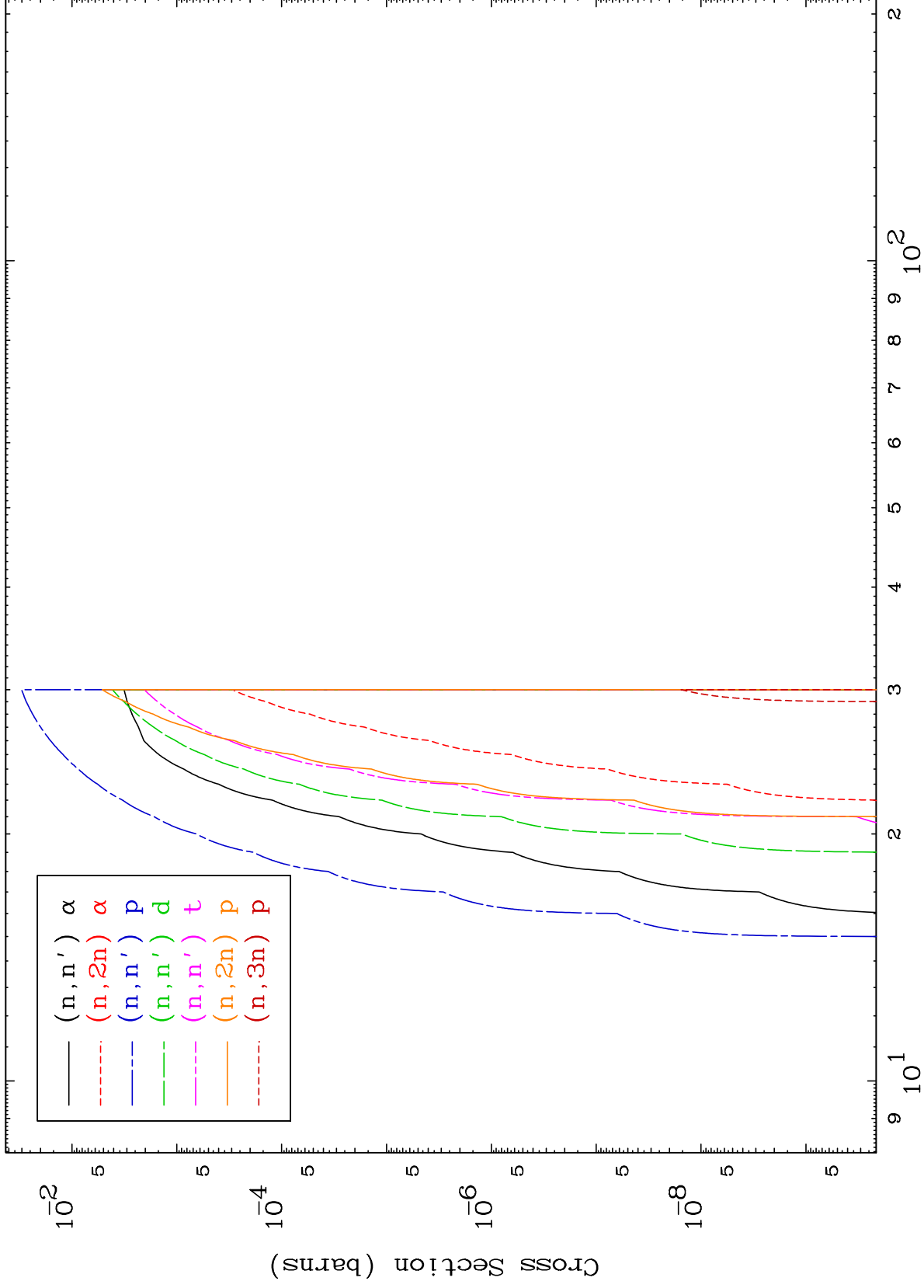
50-Sn-127



MAT 5070

Charged Particle  
293 Kelvin Cross Sections

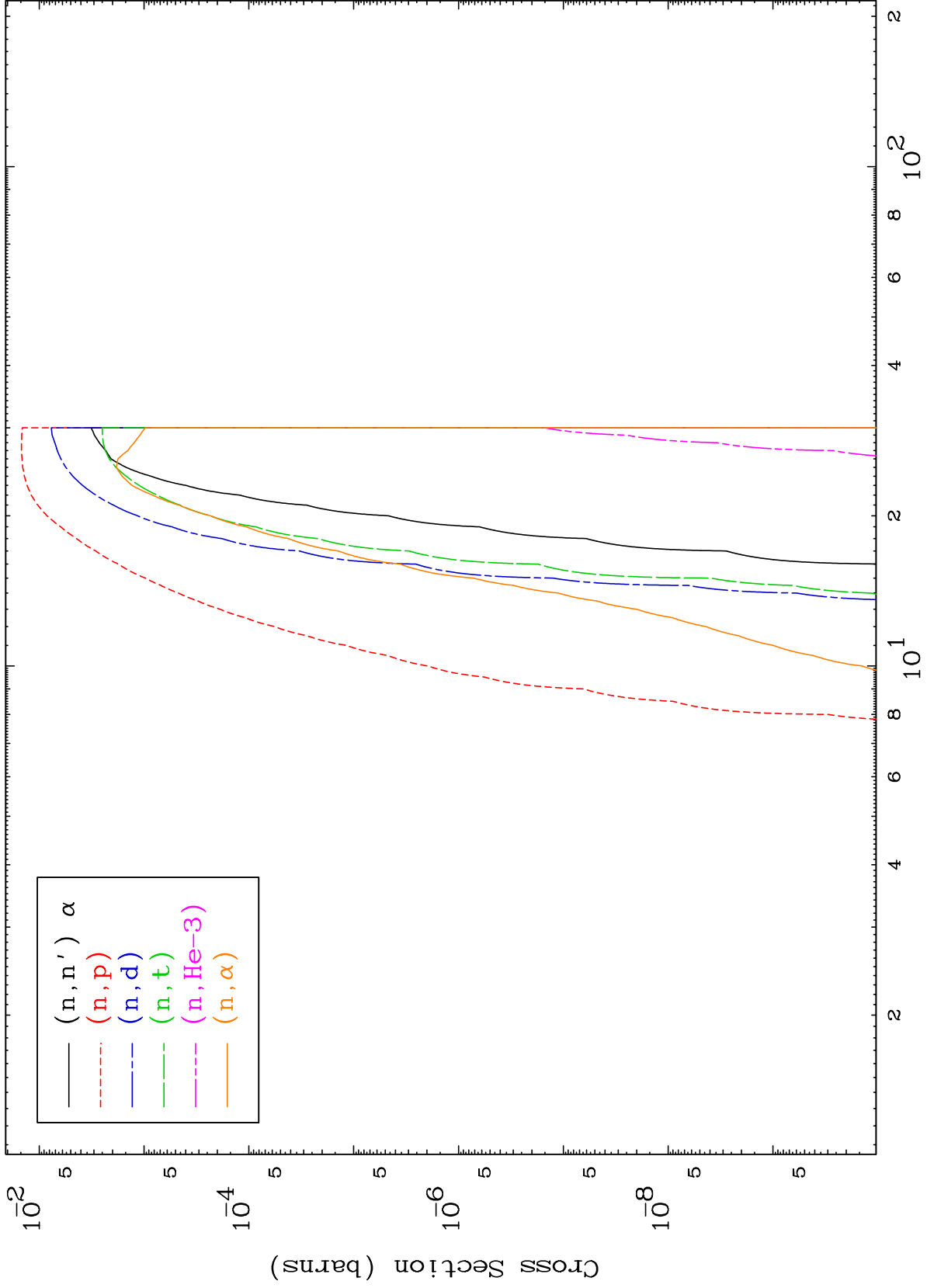
50-Sn-127



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Incident Energy (MeV)

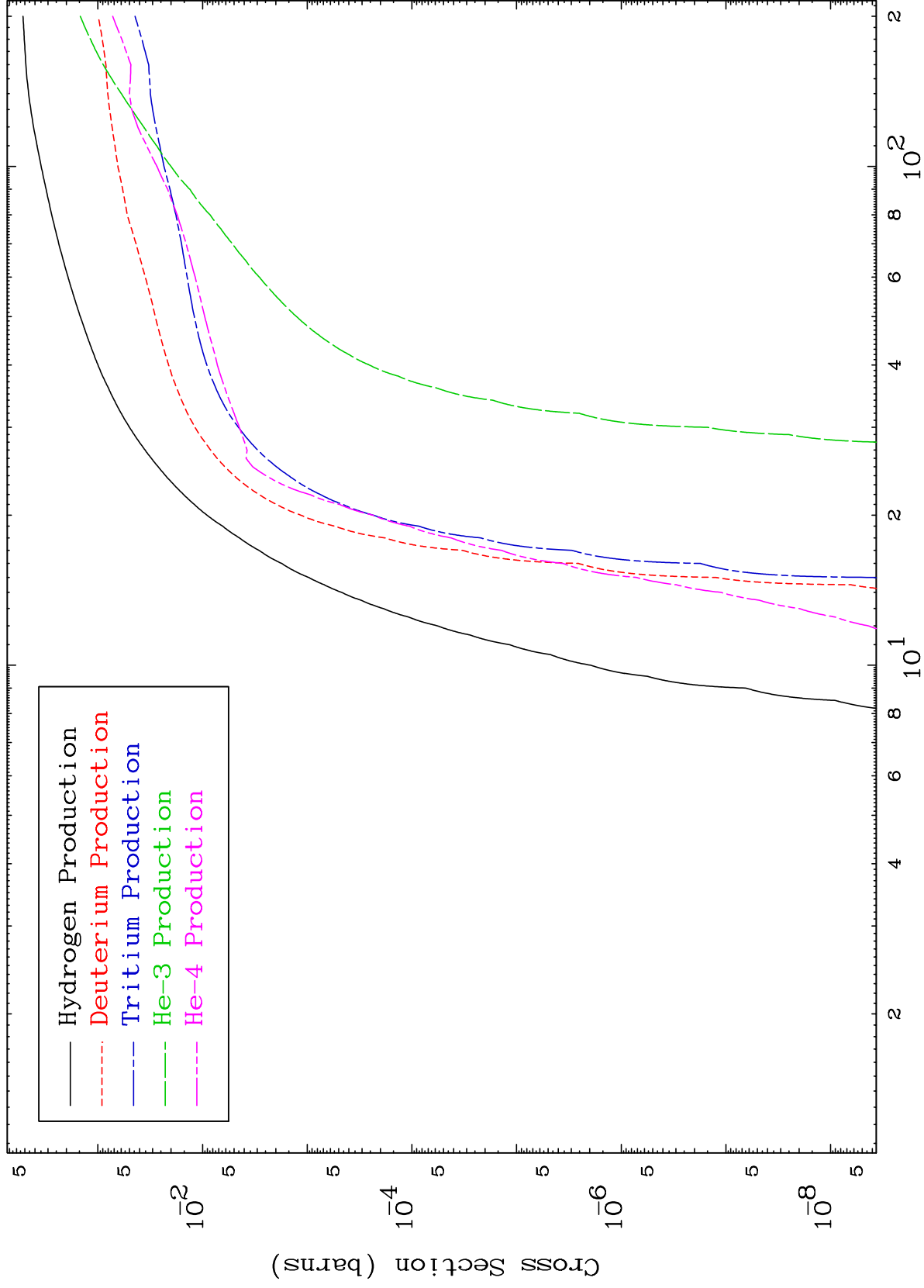
50-Sn-127



MAT 5070

Particle Production  
293 Kelvin Cross Sections

50-Sn-127



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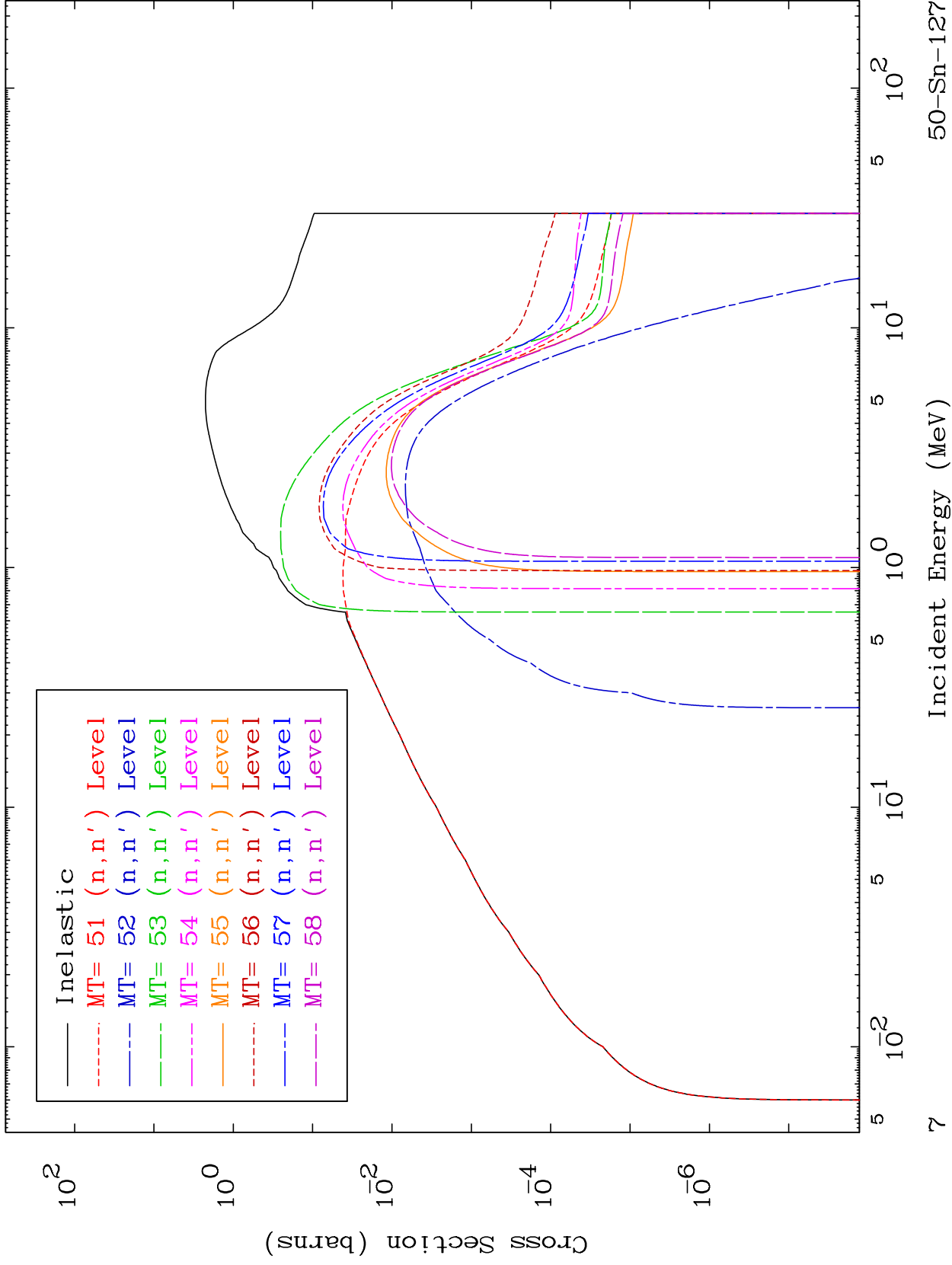
Incident Energy (MeV)

50-Sn-127

MAT 5070

(n,n') Levels  
293 Kelvin Cross Sections

50-Sn-127



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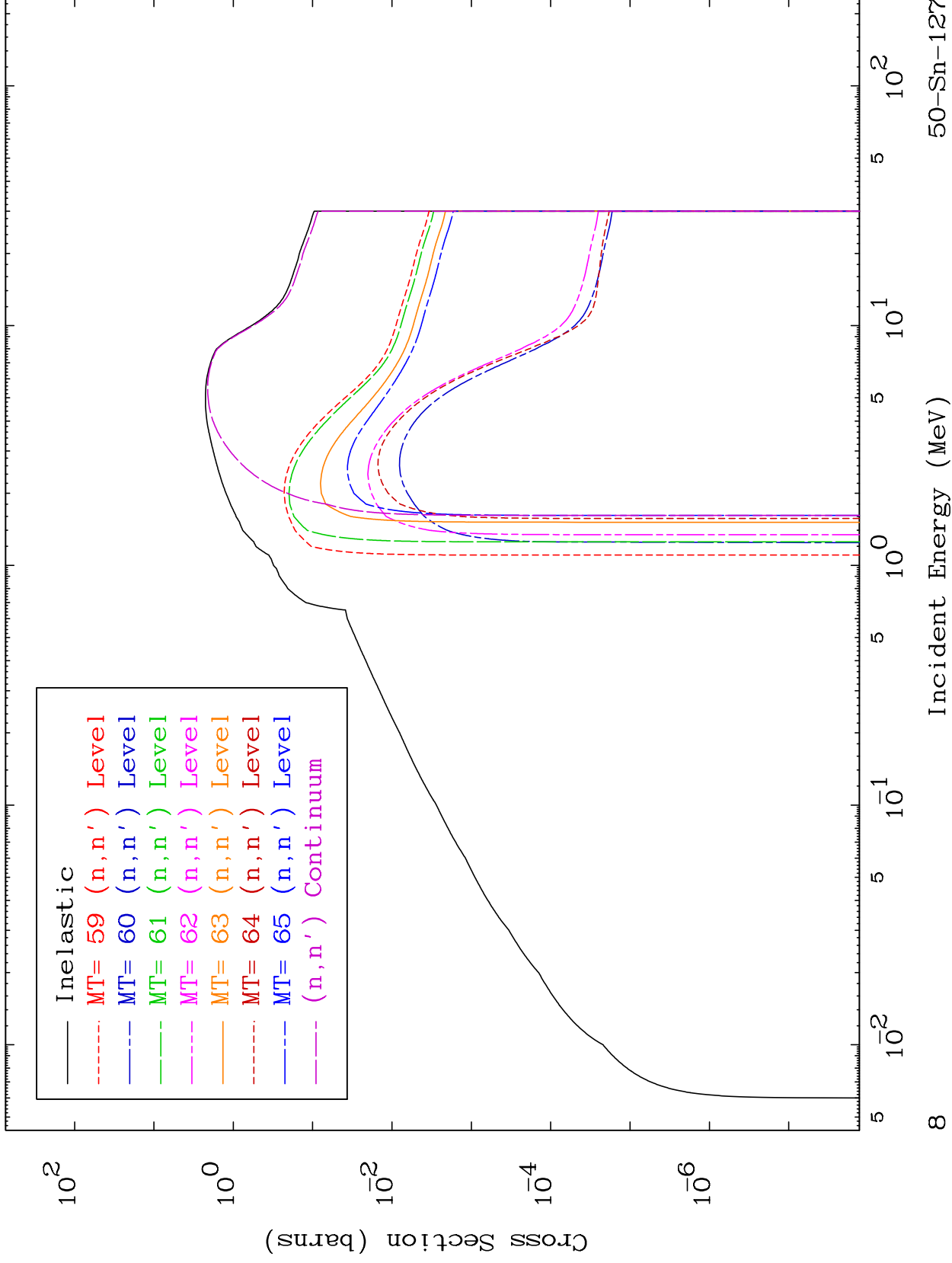
50-Sn-127



MAT 5070

293 Kelvin Cross Sections  
(n,n') Levels

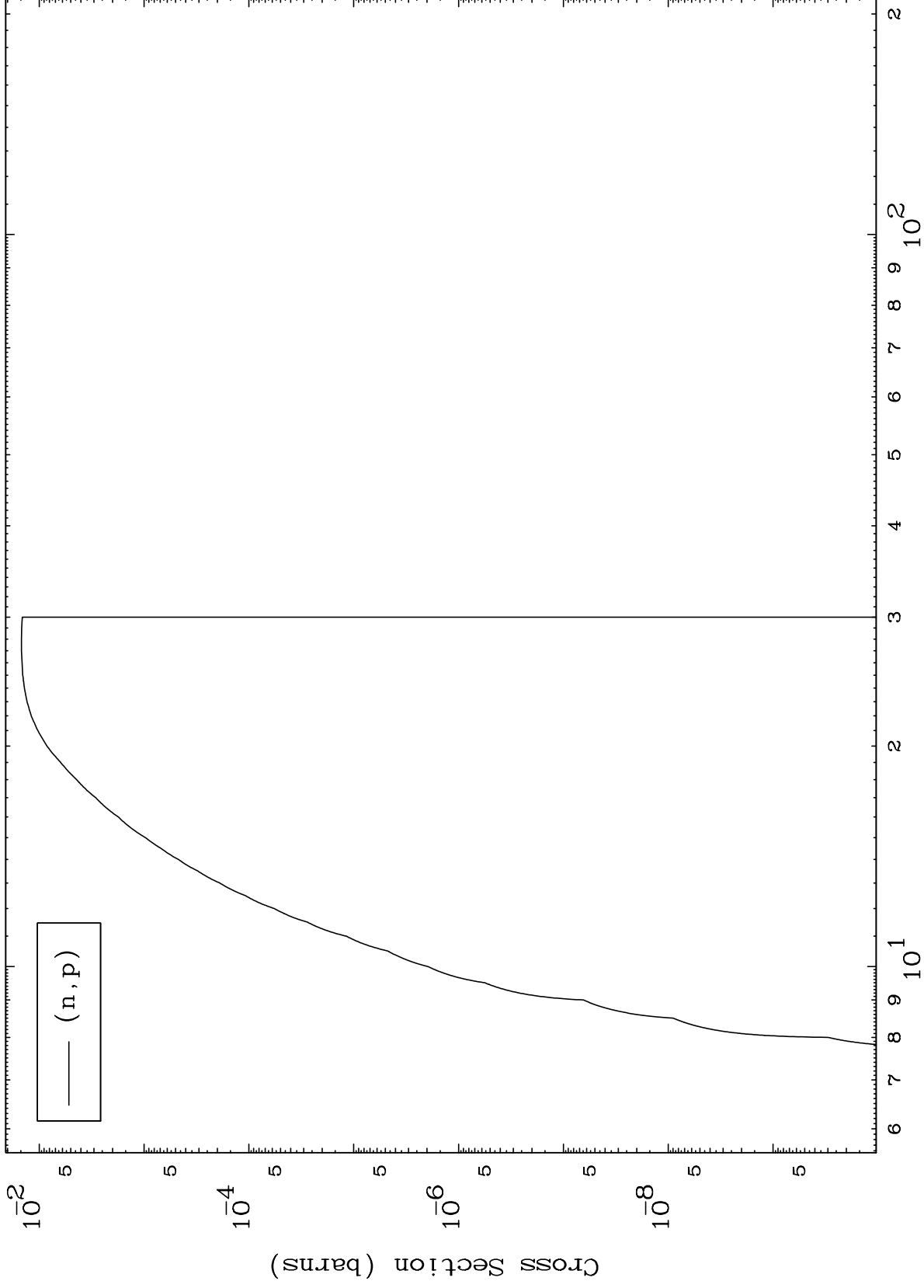
50-Sn-127



MAT 5070

(n,p) Levels  
293 Kelvin Cross Sections

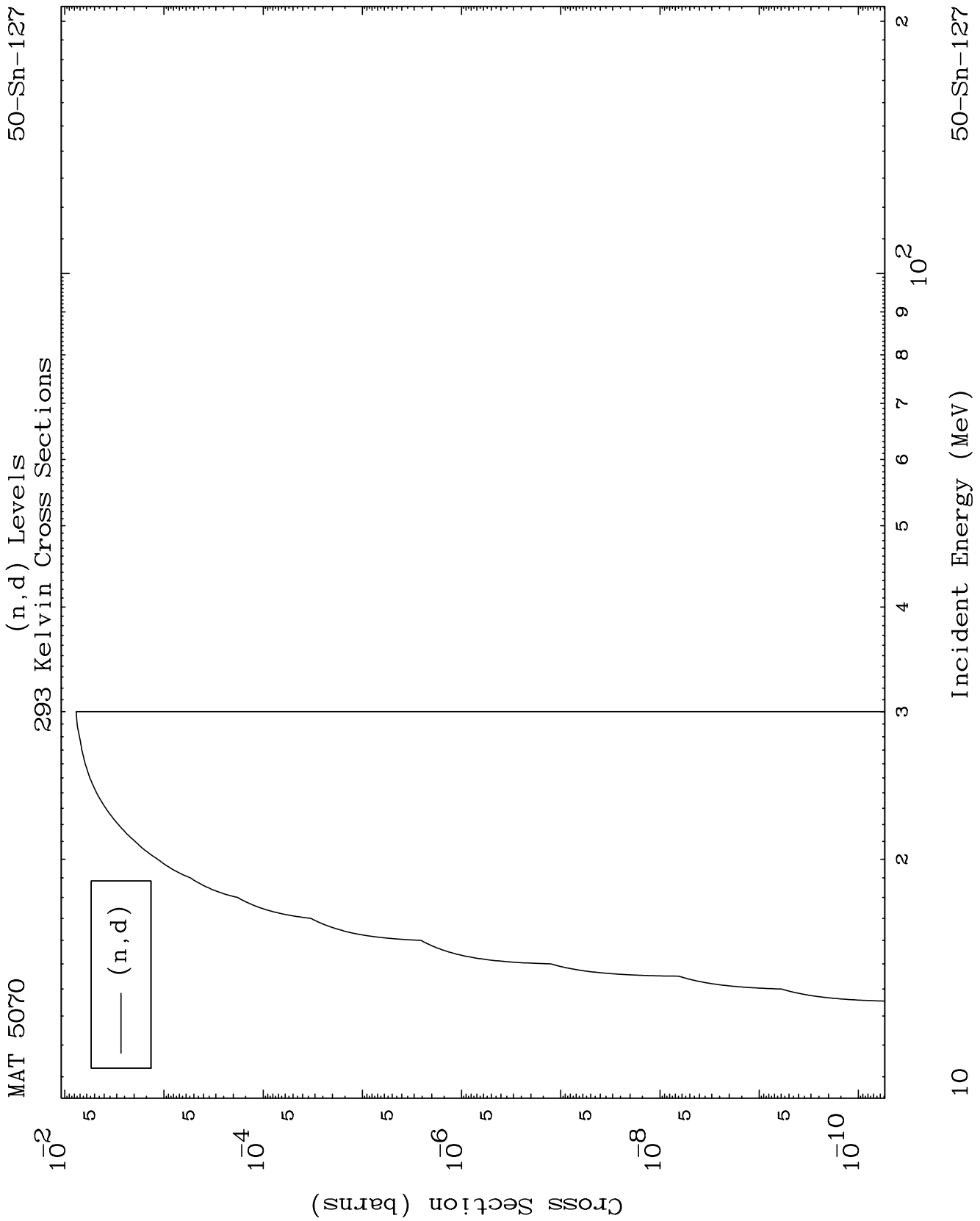
50-Sn-127



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Incident Energy (MeV)

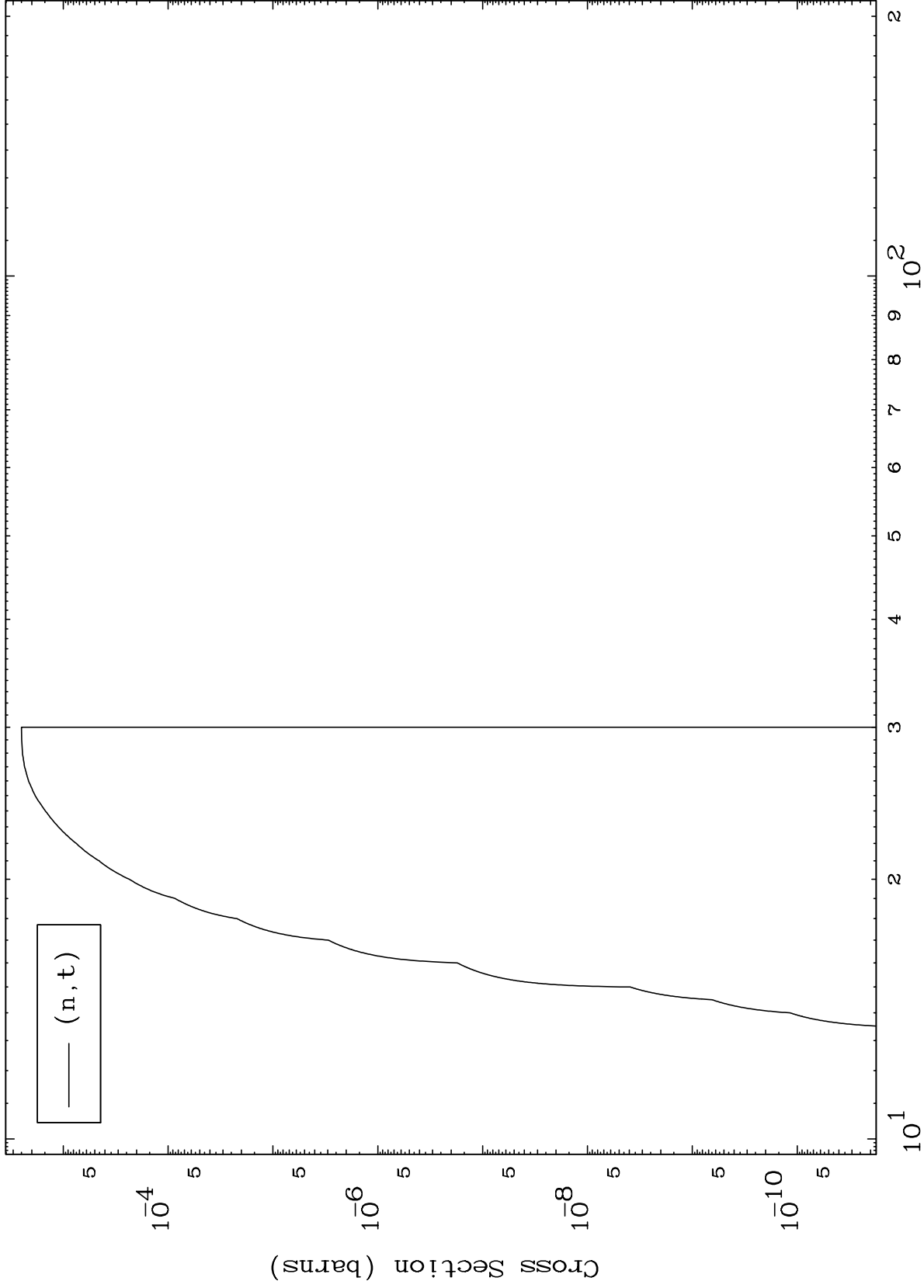
50-Sn-127



MAT 5070

(n,t) Levels  
293 Kelvin Cross Sections

50-Sn-127



Incident Energy (MeV)

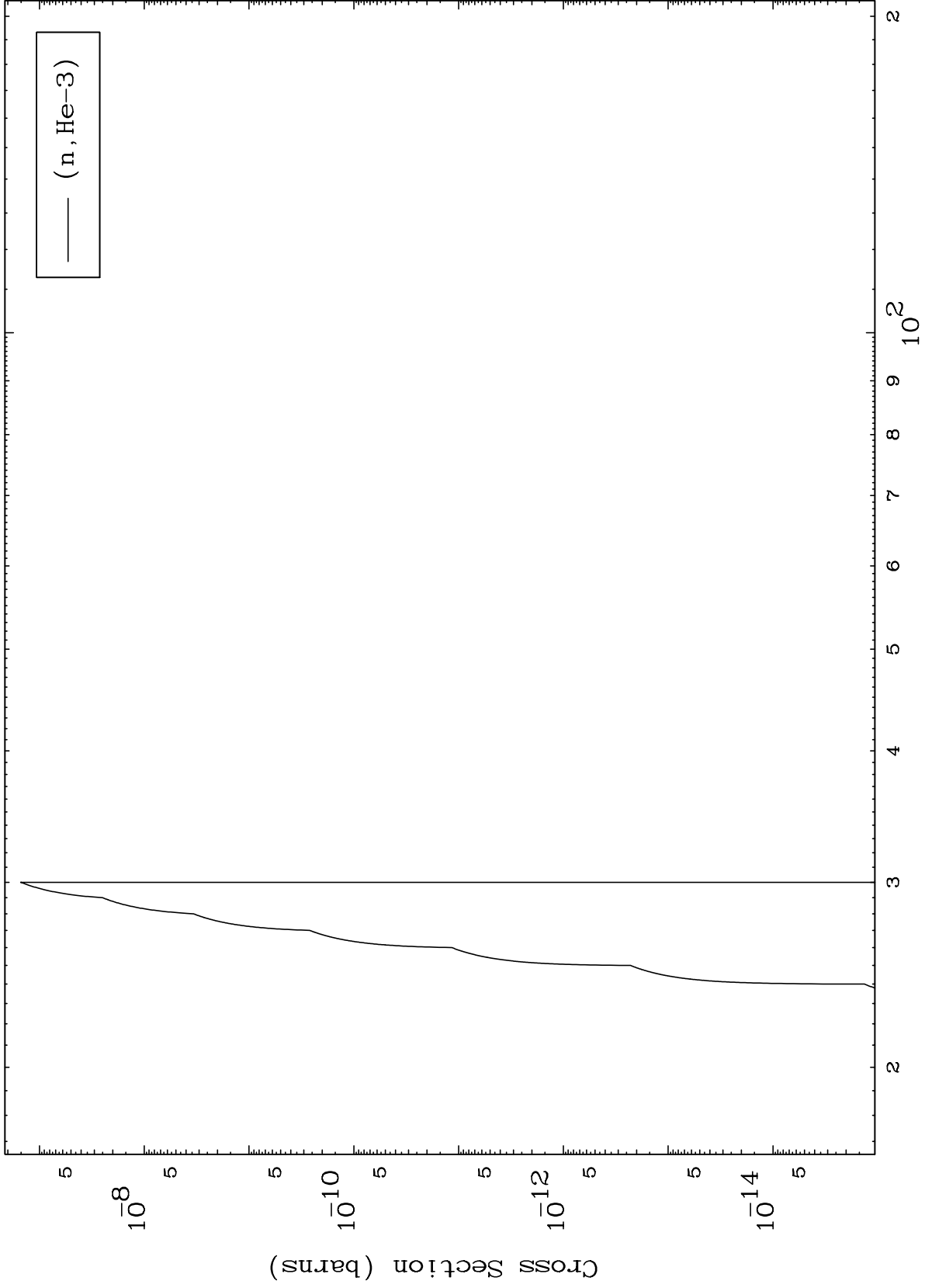
50-Sn-127

11

MAT 5070

(n,He3) Levels  
293 Kelvin Cross Sections

50-Sn-127



12

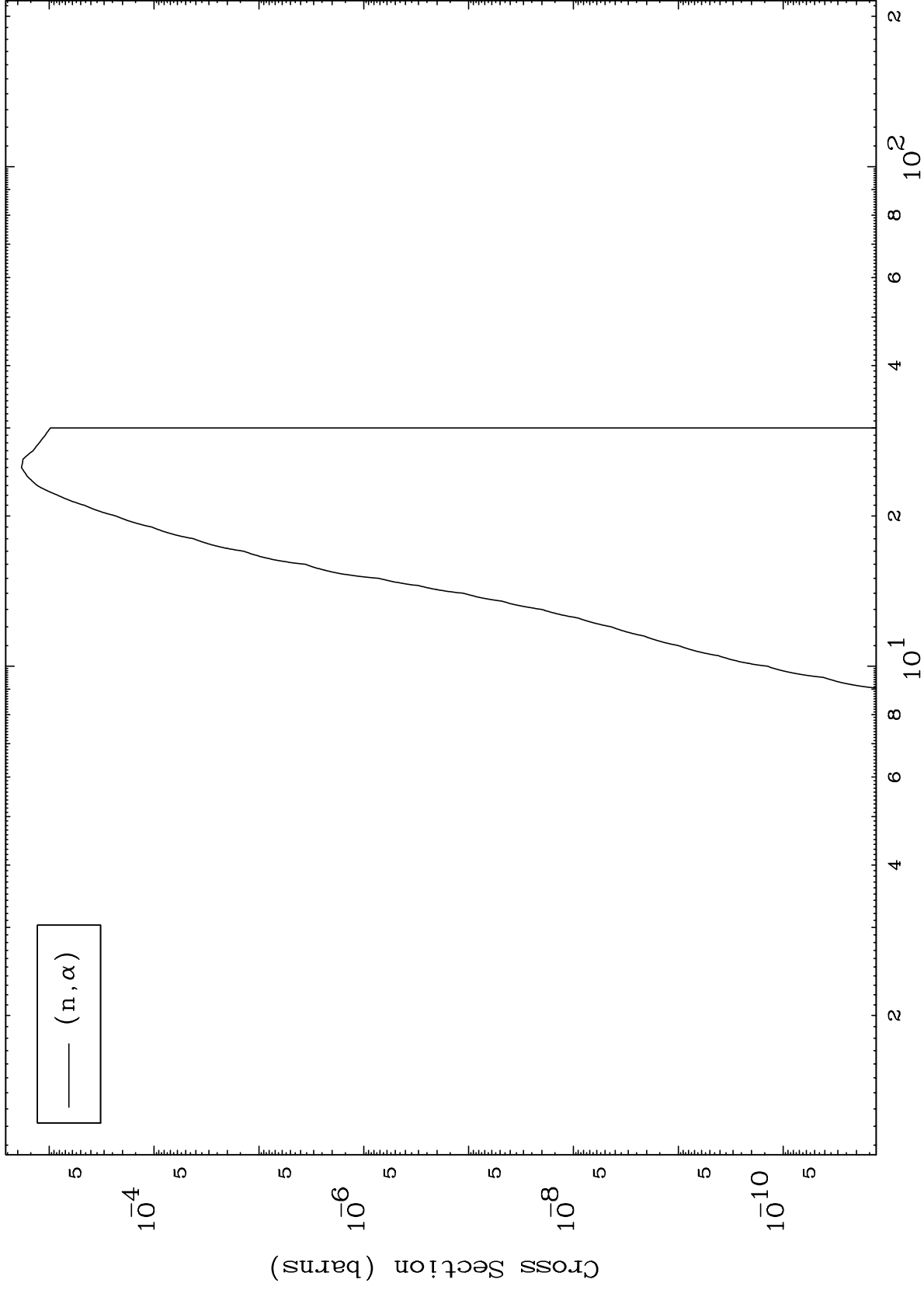
Incident Energy (MeV)

50-Sn-127

MAT 5070

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

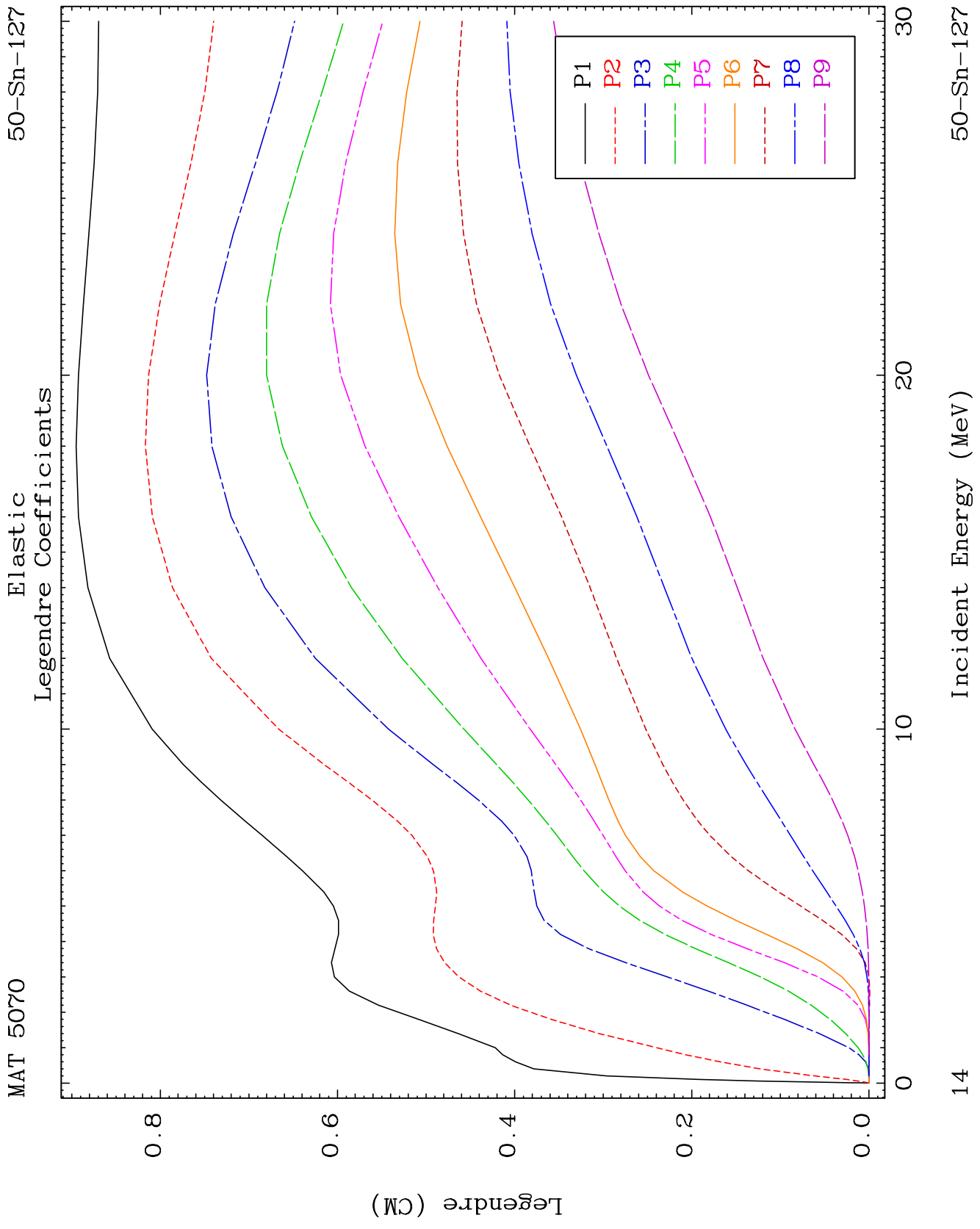
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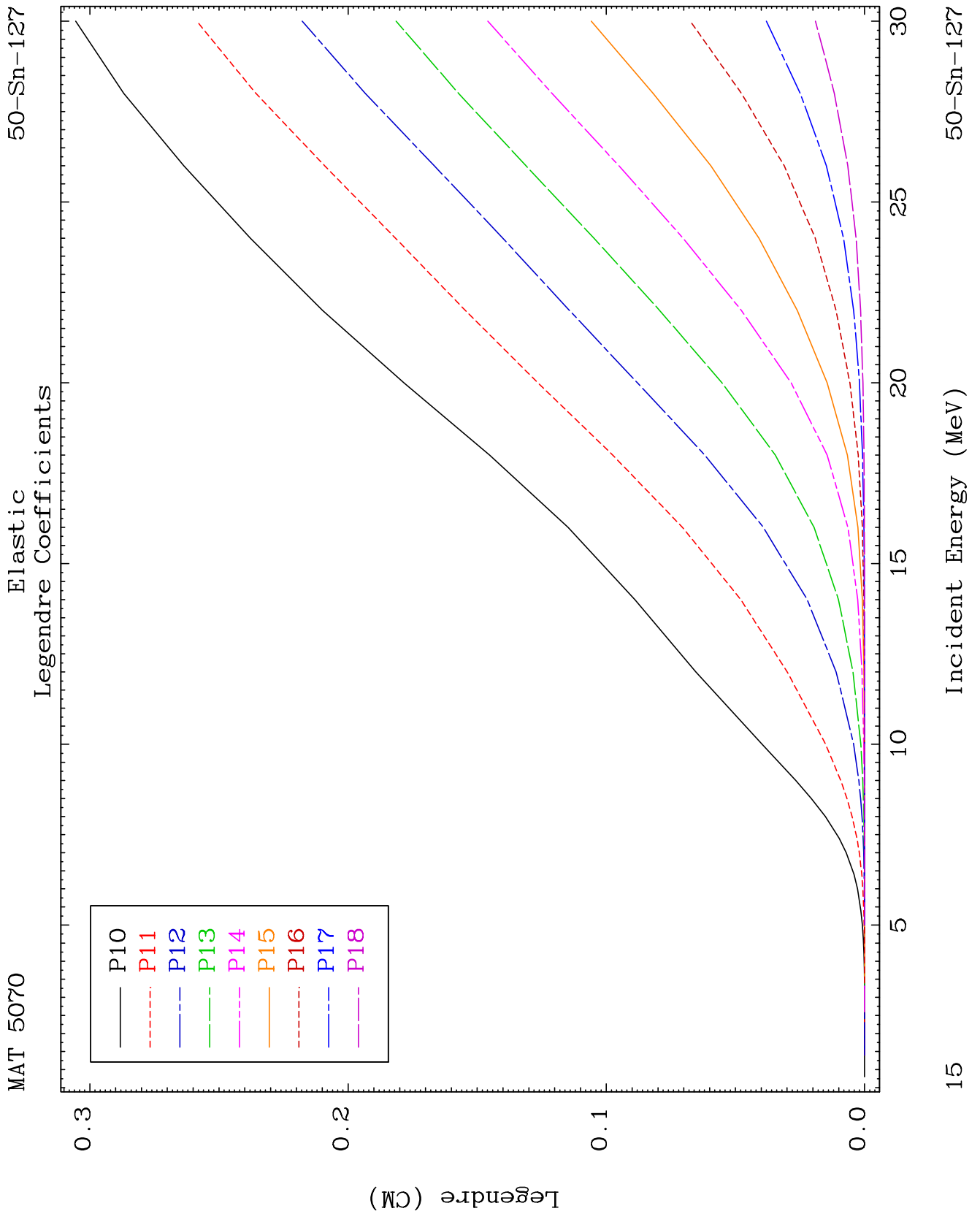


13

Incident Energy (MeV)

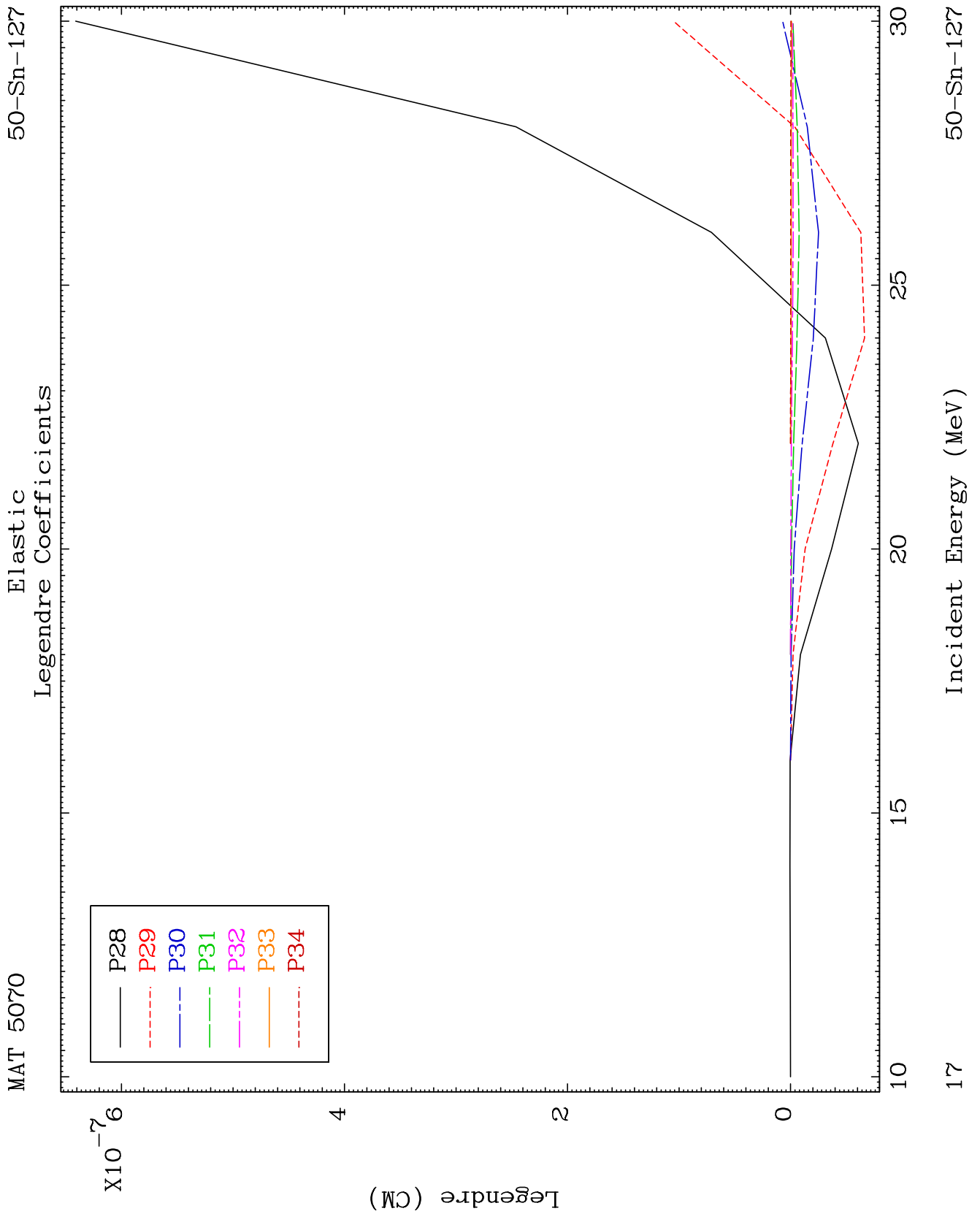
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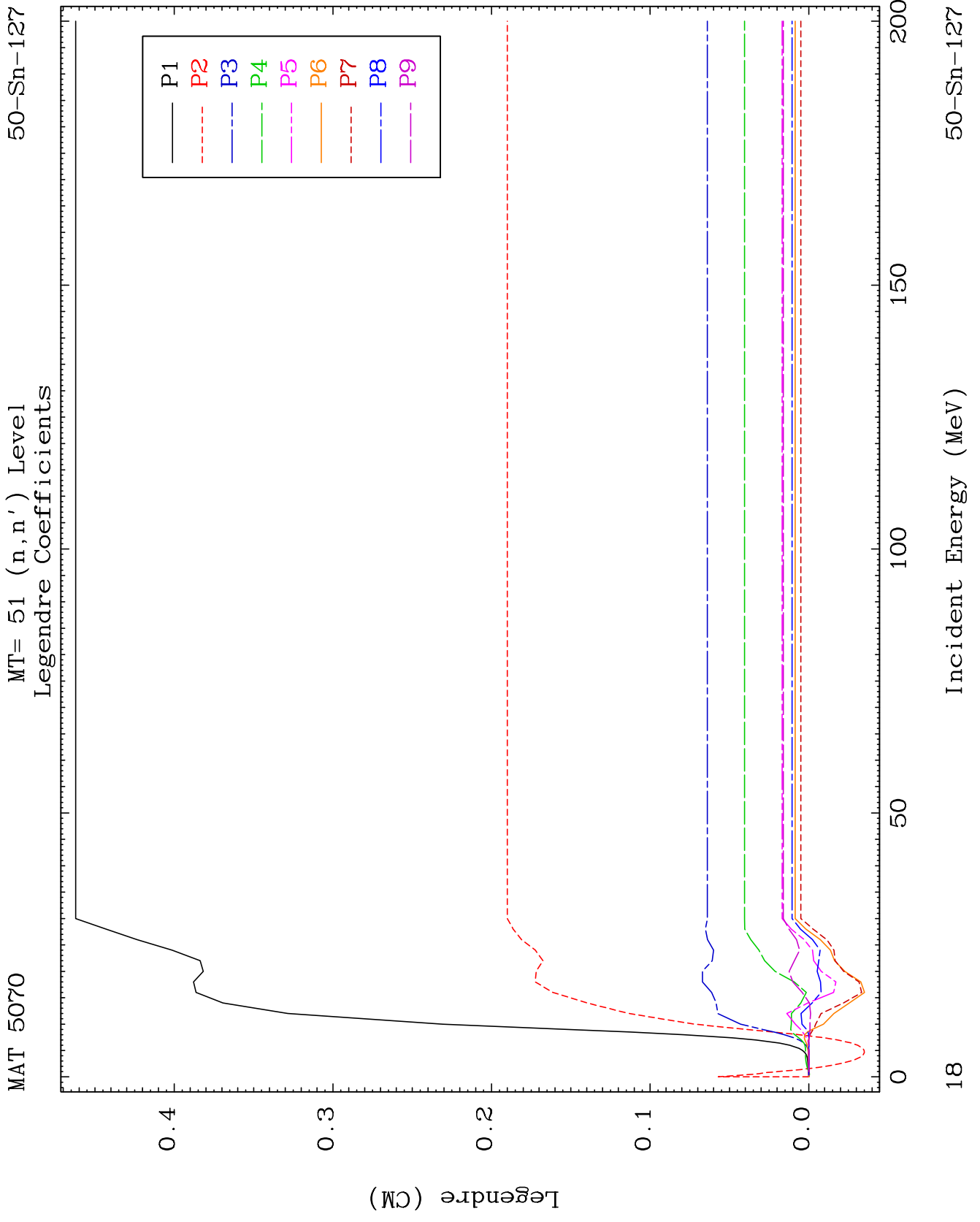








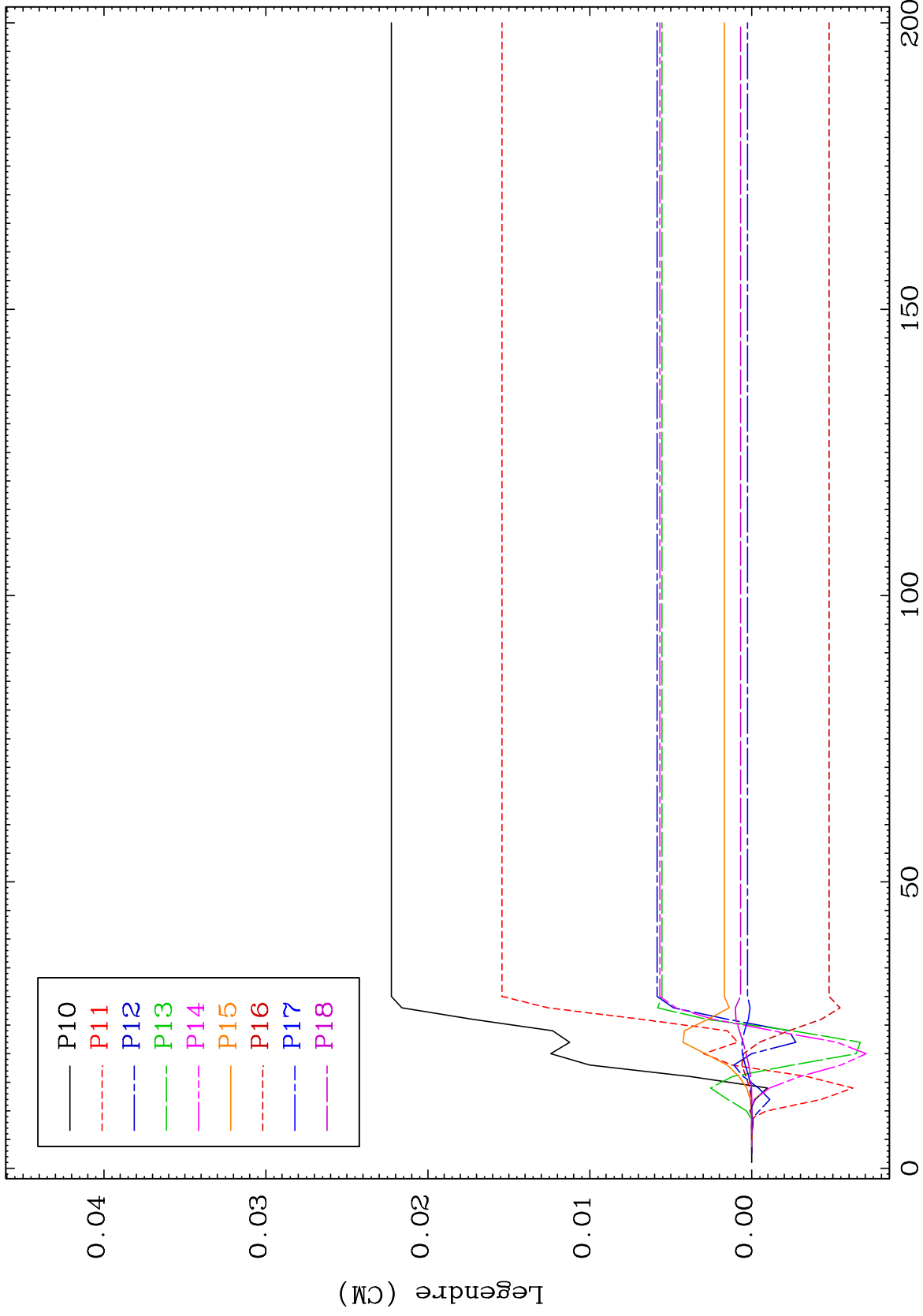




MAT 5070

MT= 51 (n,n') Level  
Legendre Coefficients

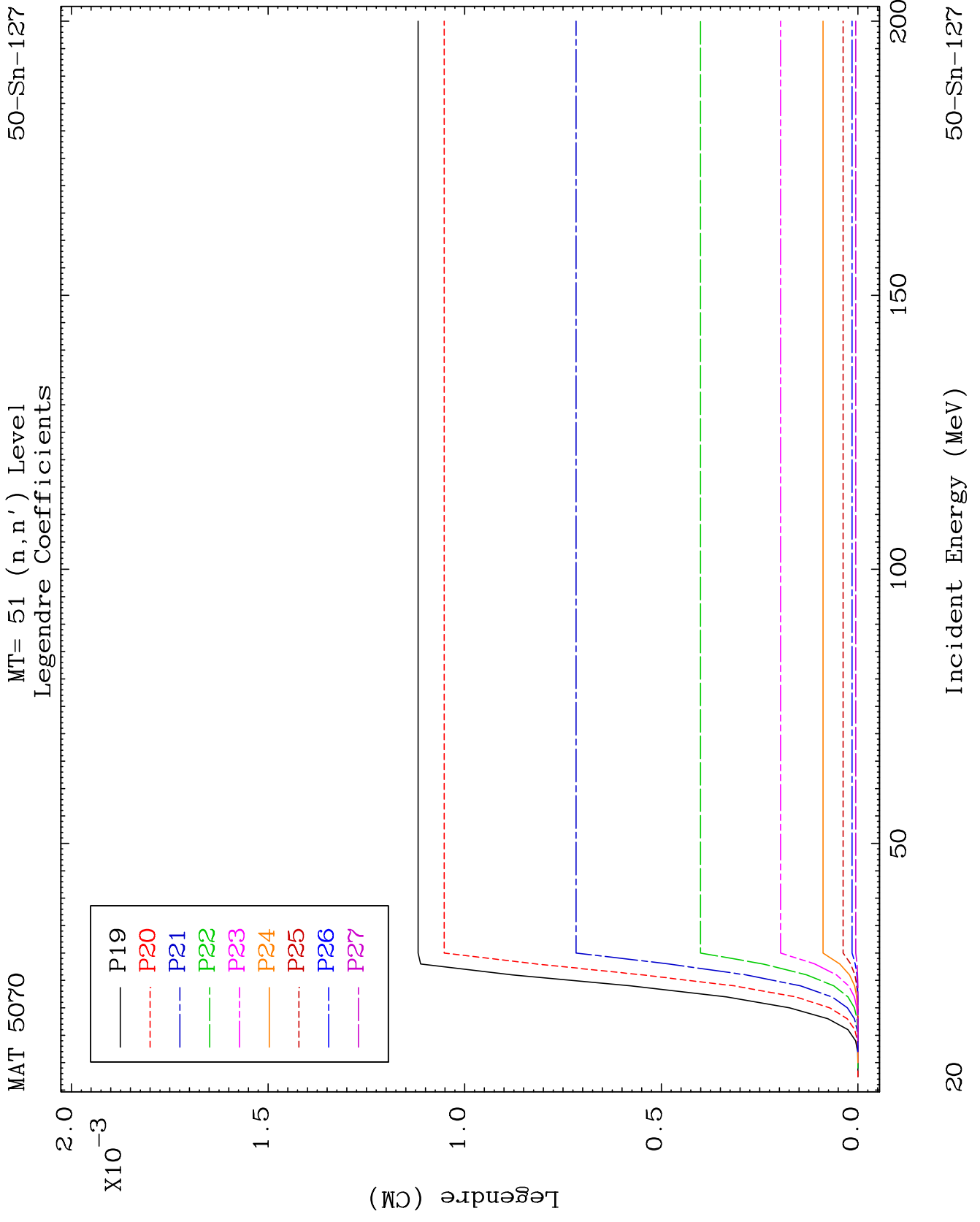
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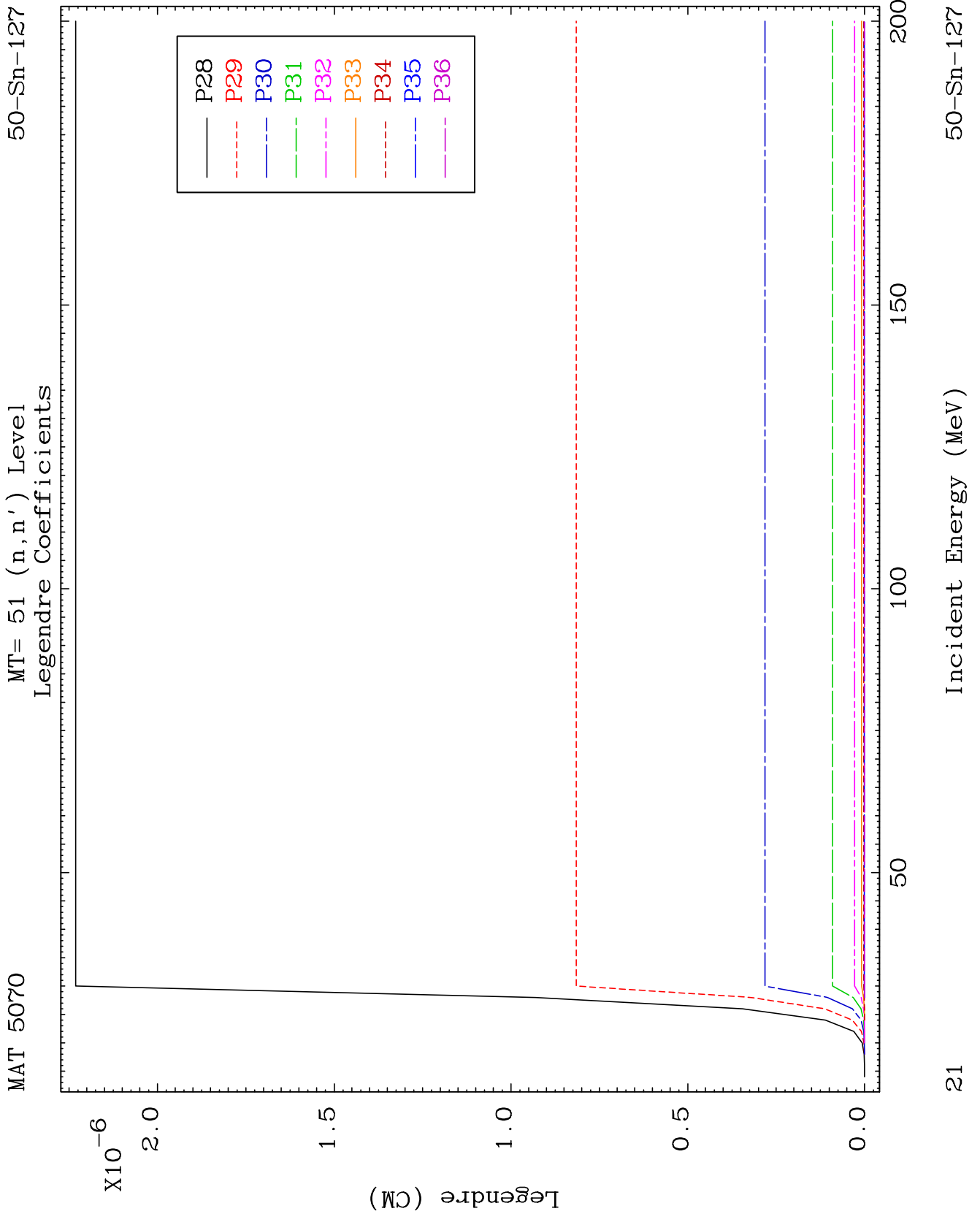


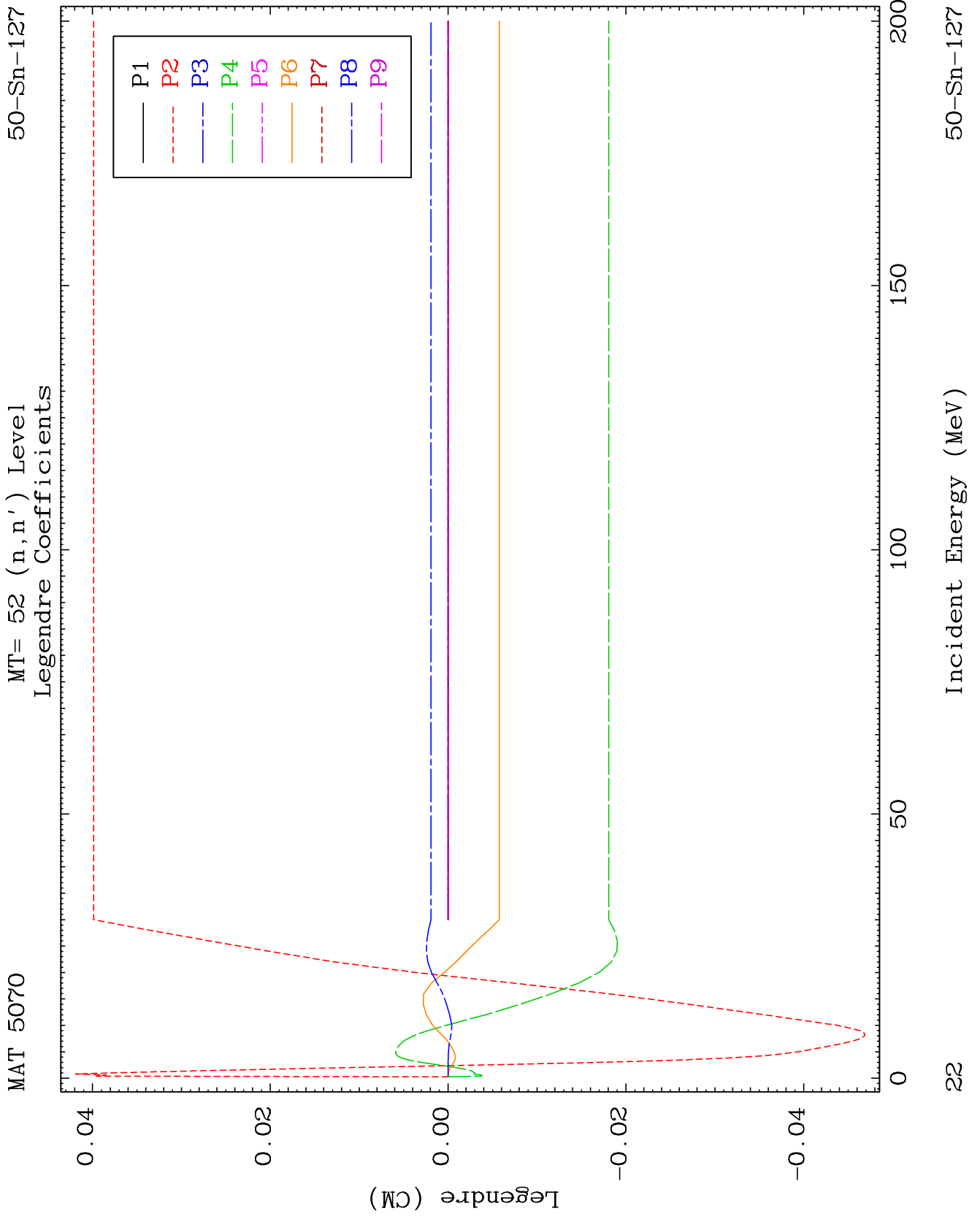
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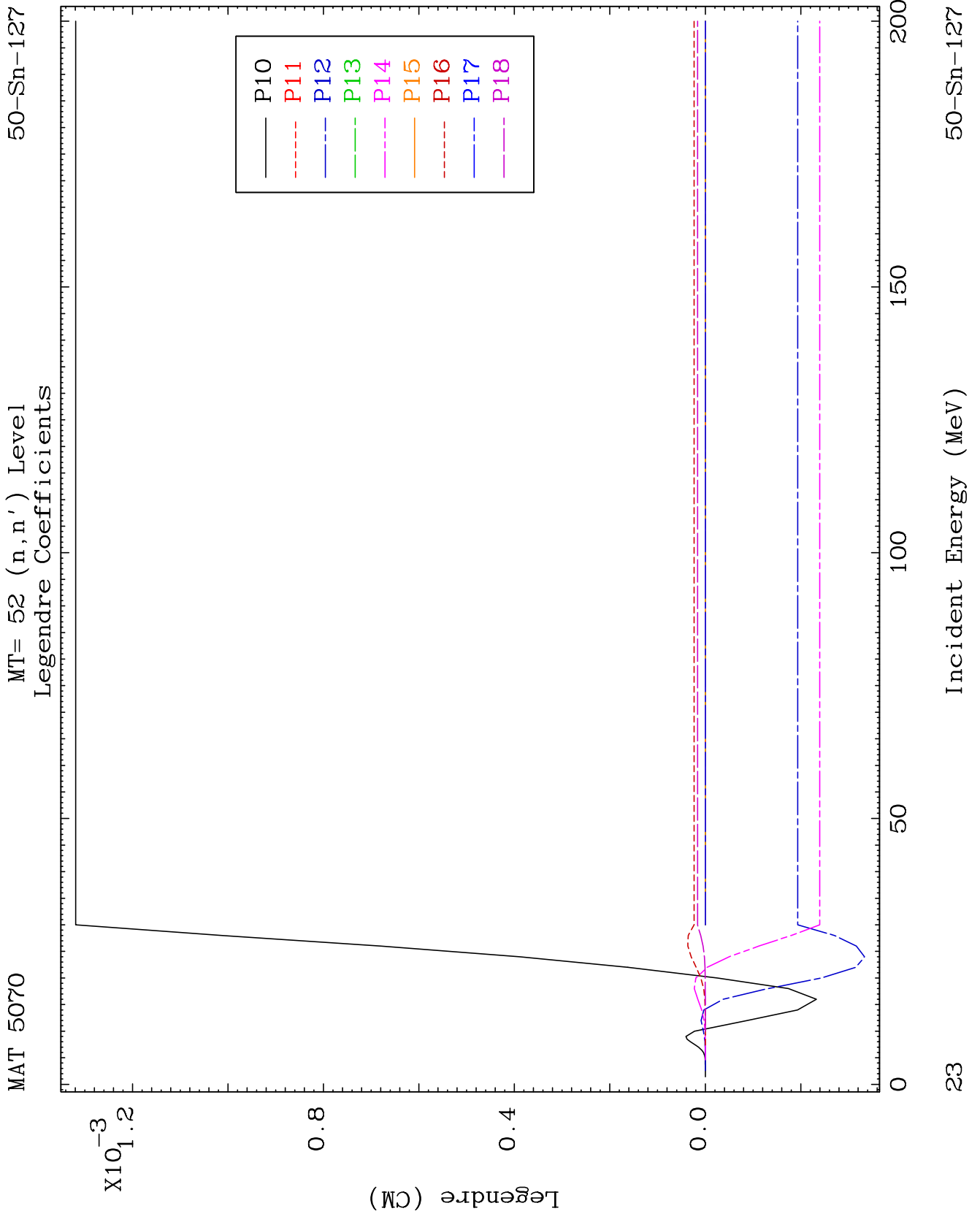
Incident Energy (MeV)

50-Sn-127

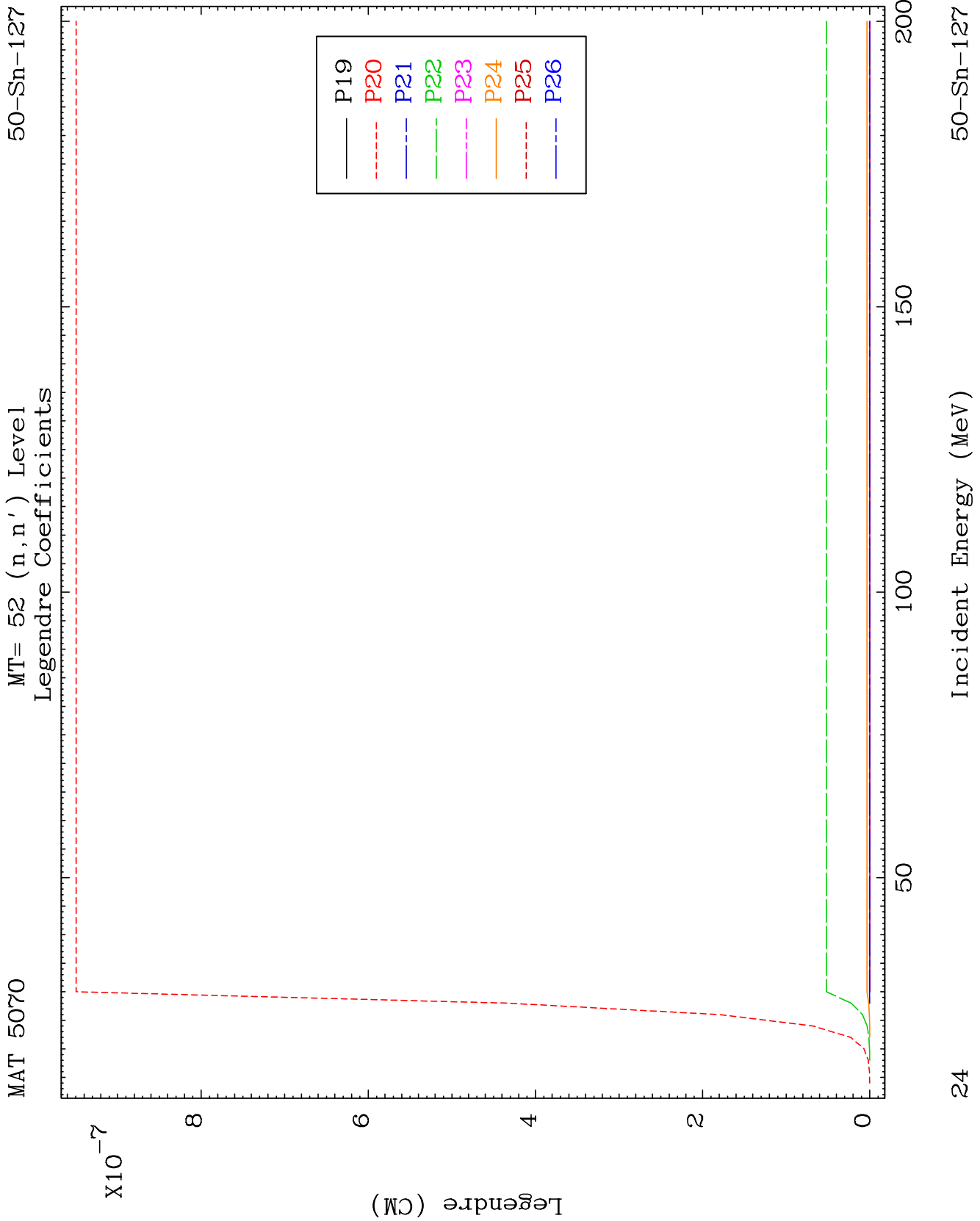




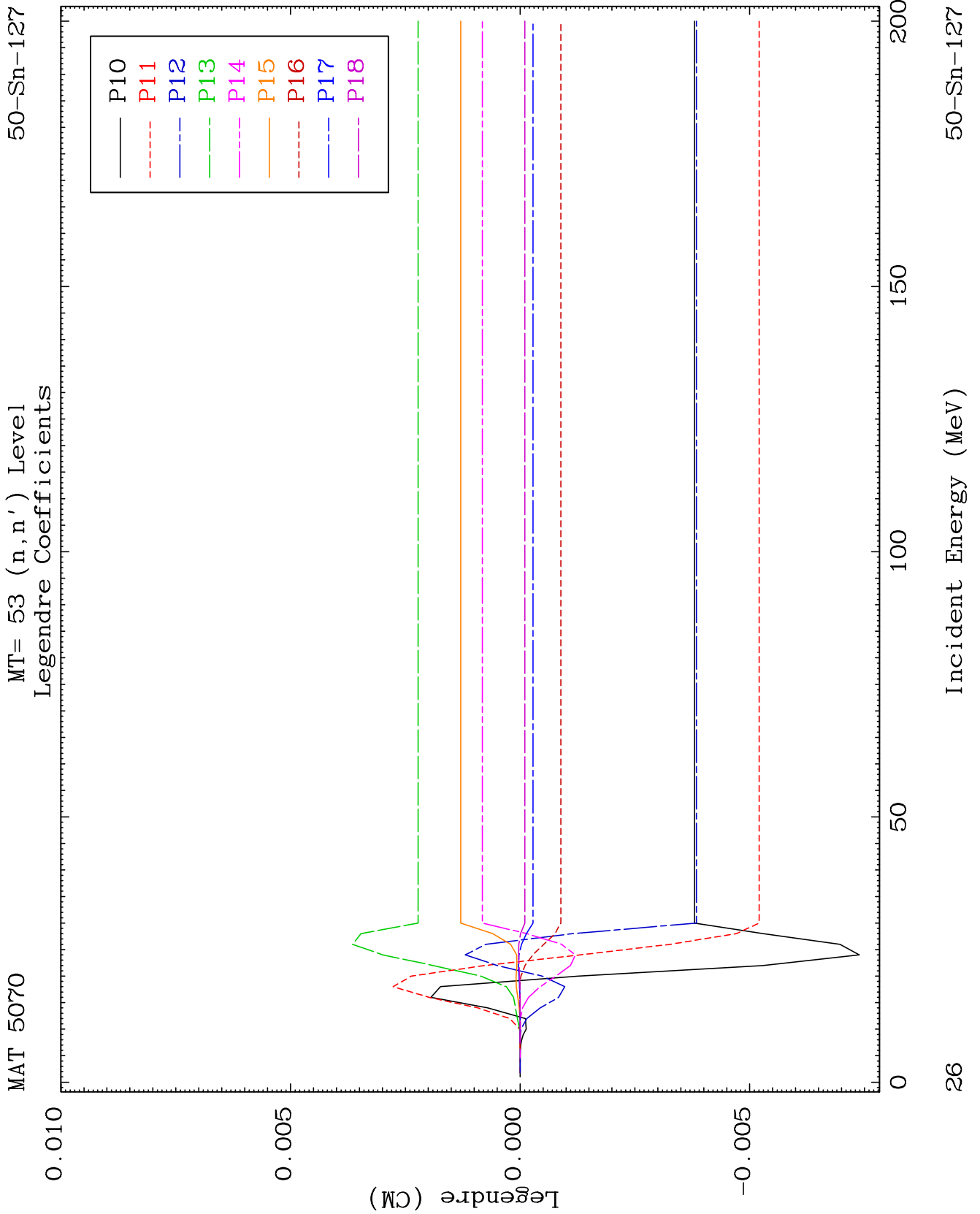


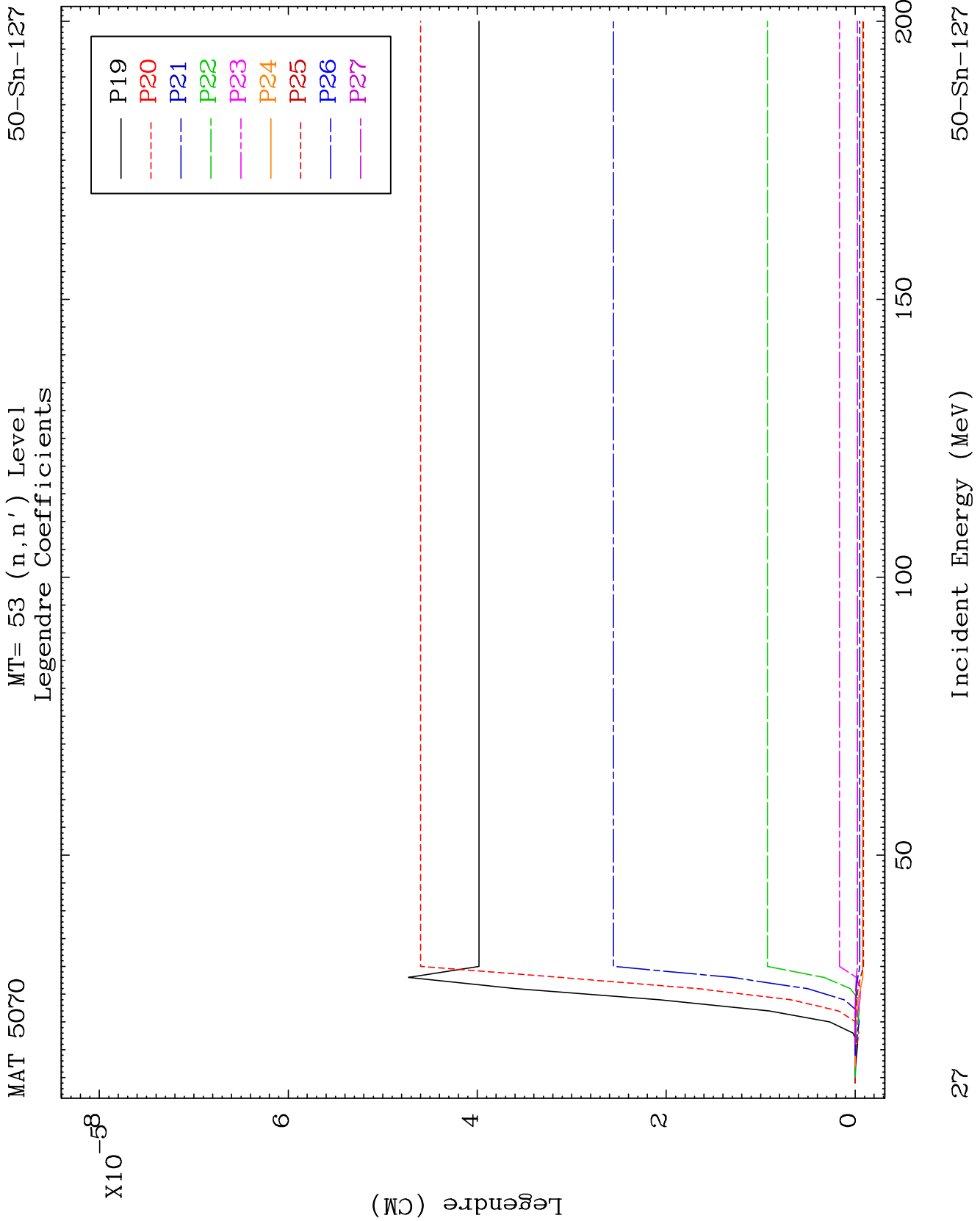


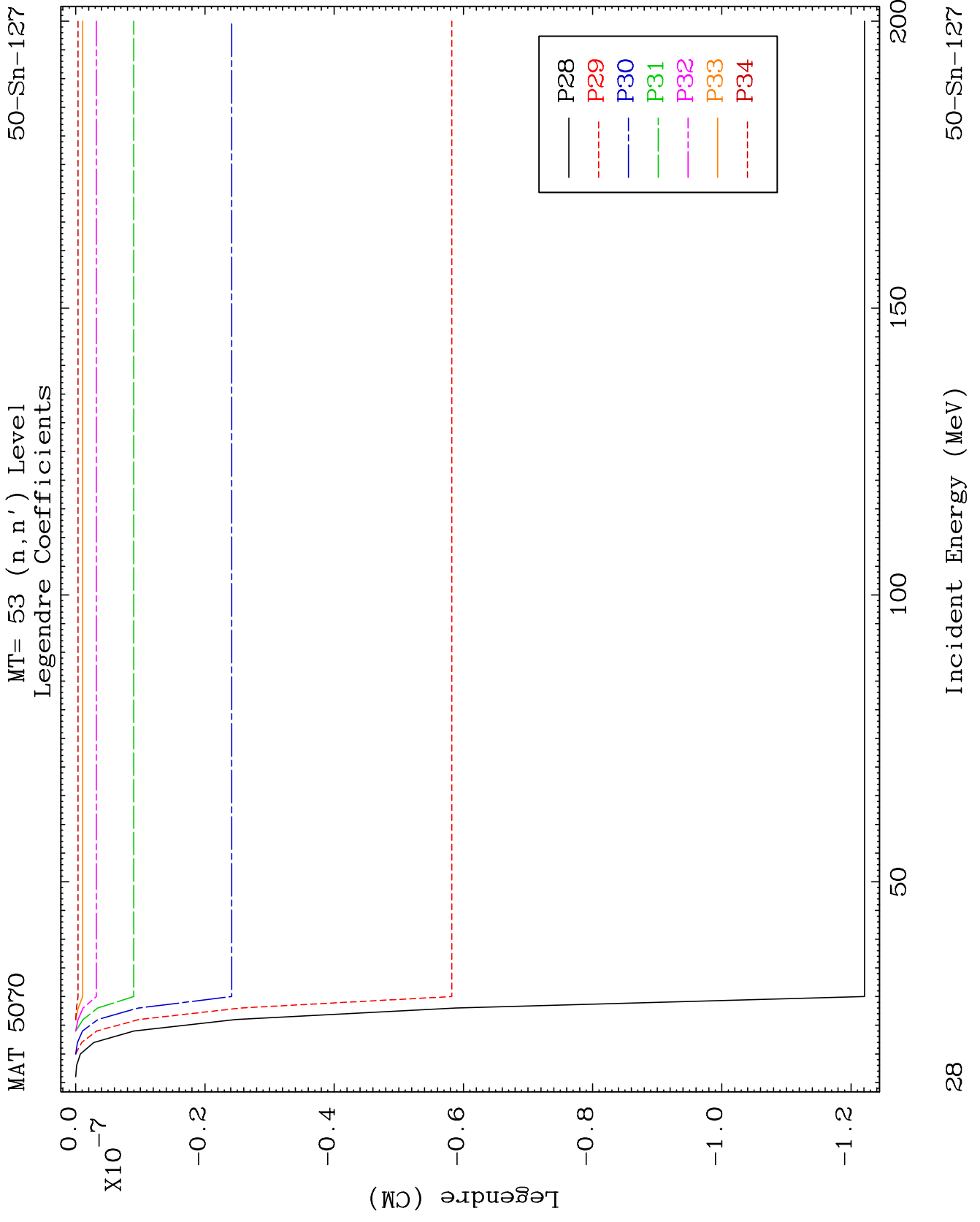








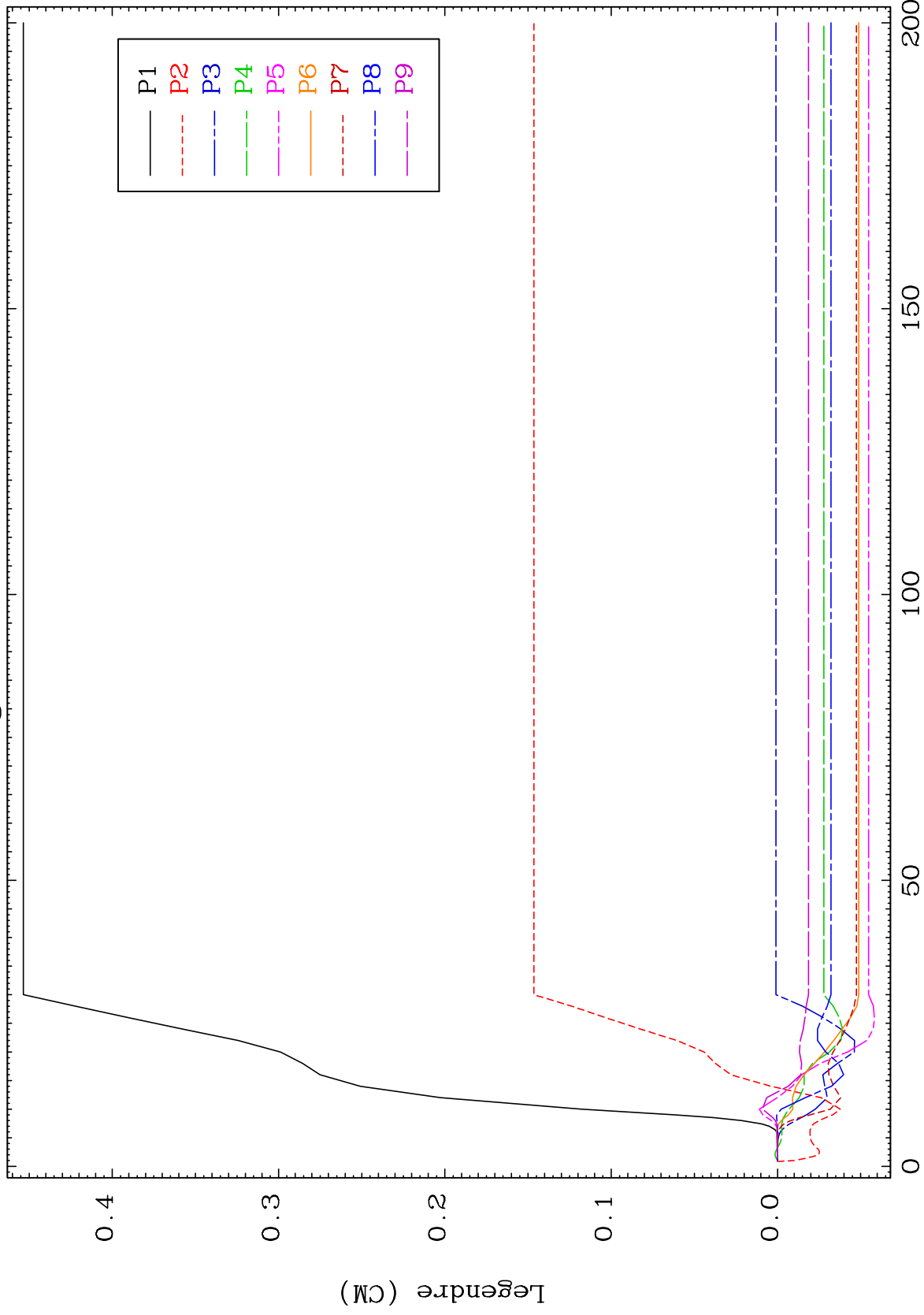




MAT 5070

MT= 54 (n,n') Level  
Legendre Coefficients

50-Sn-127

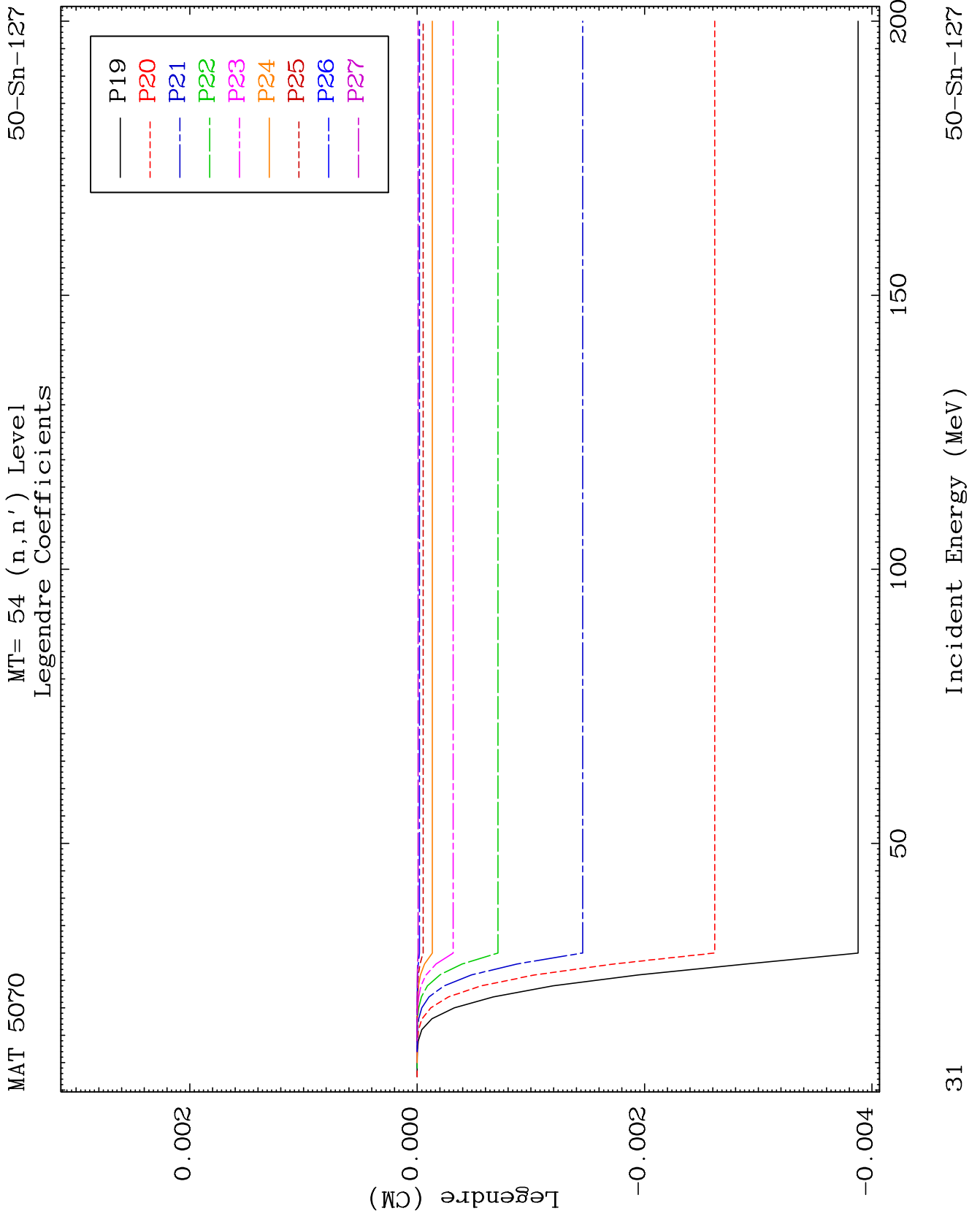


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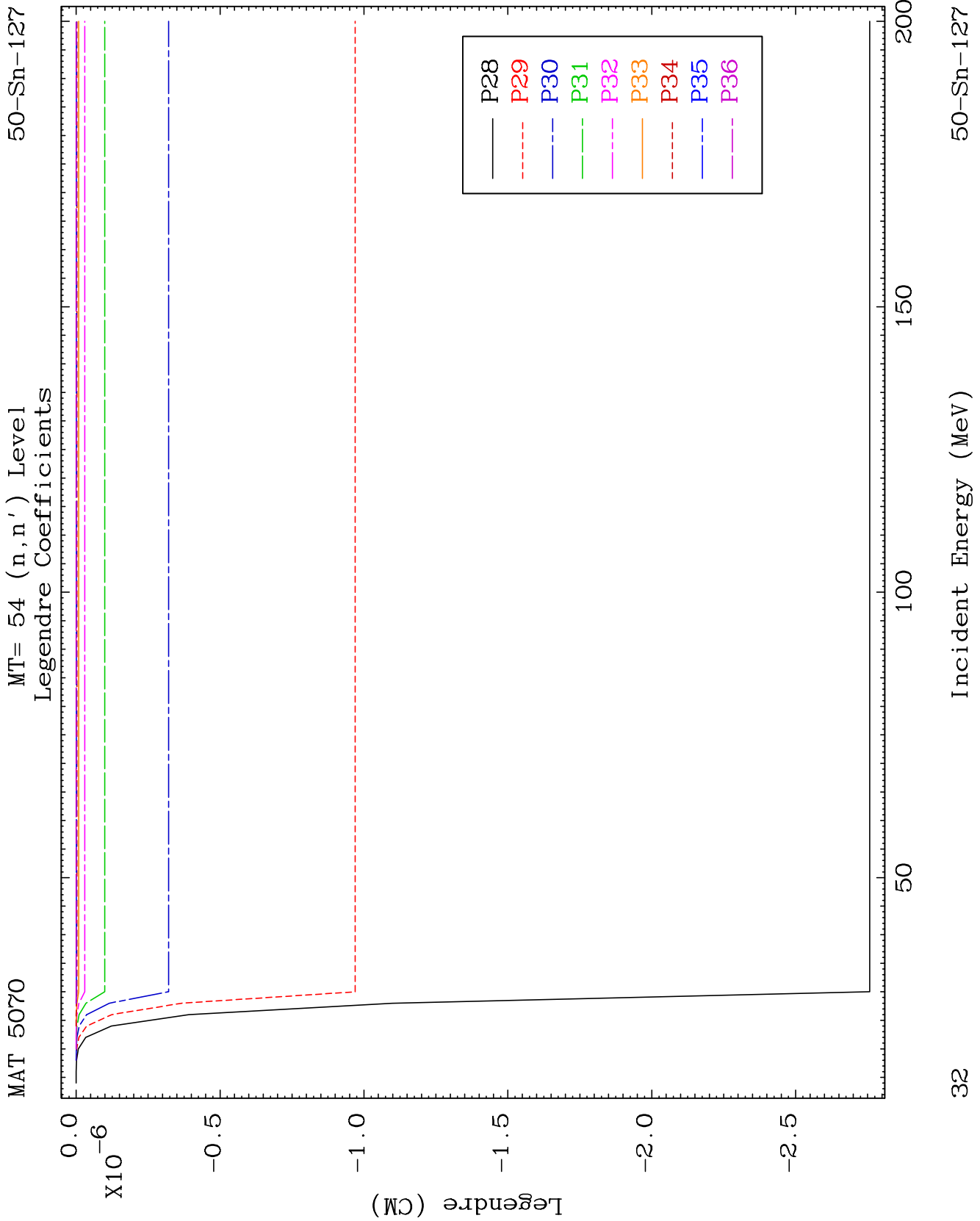
Incident Energy (MeV)

50-Sn-127

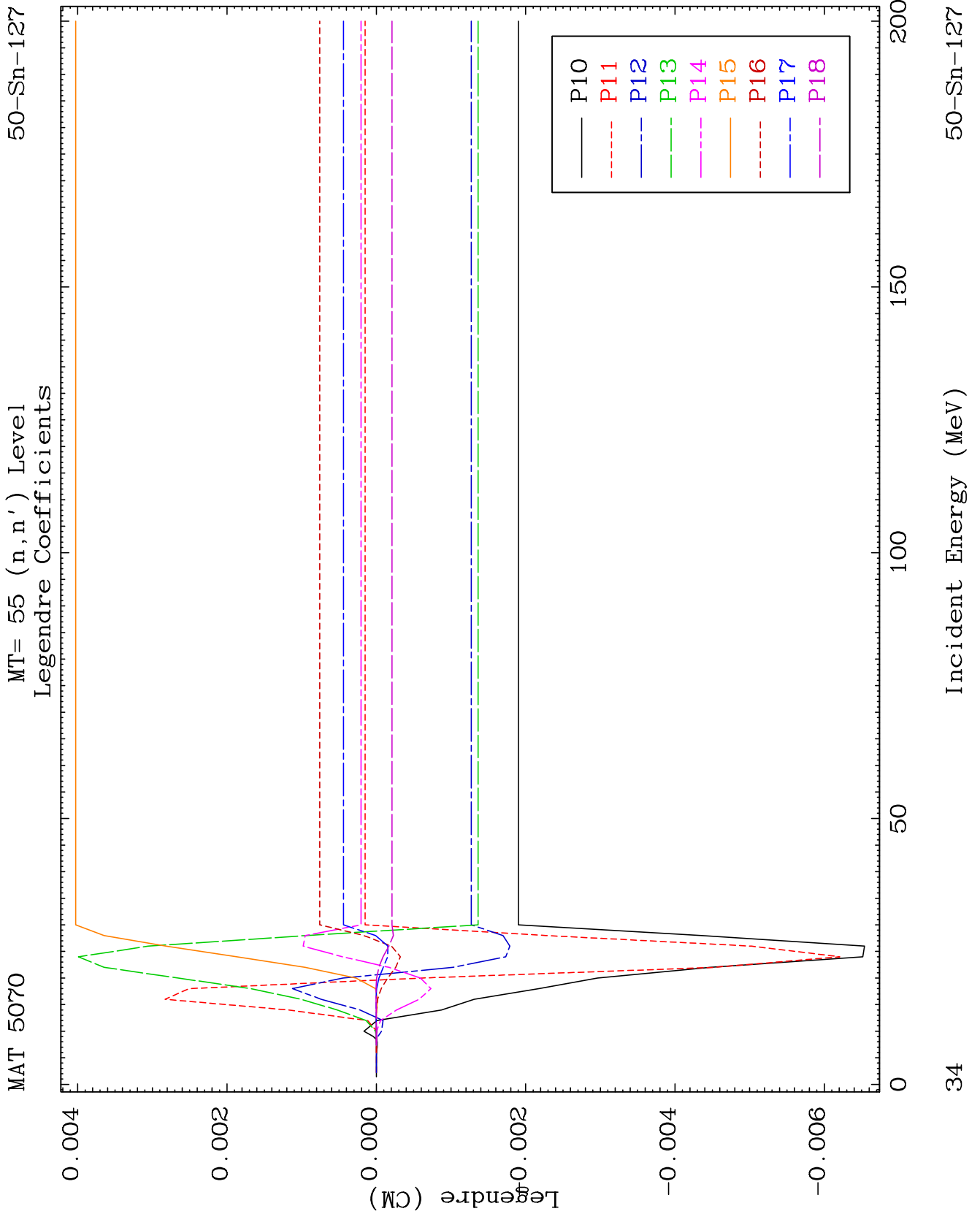


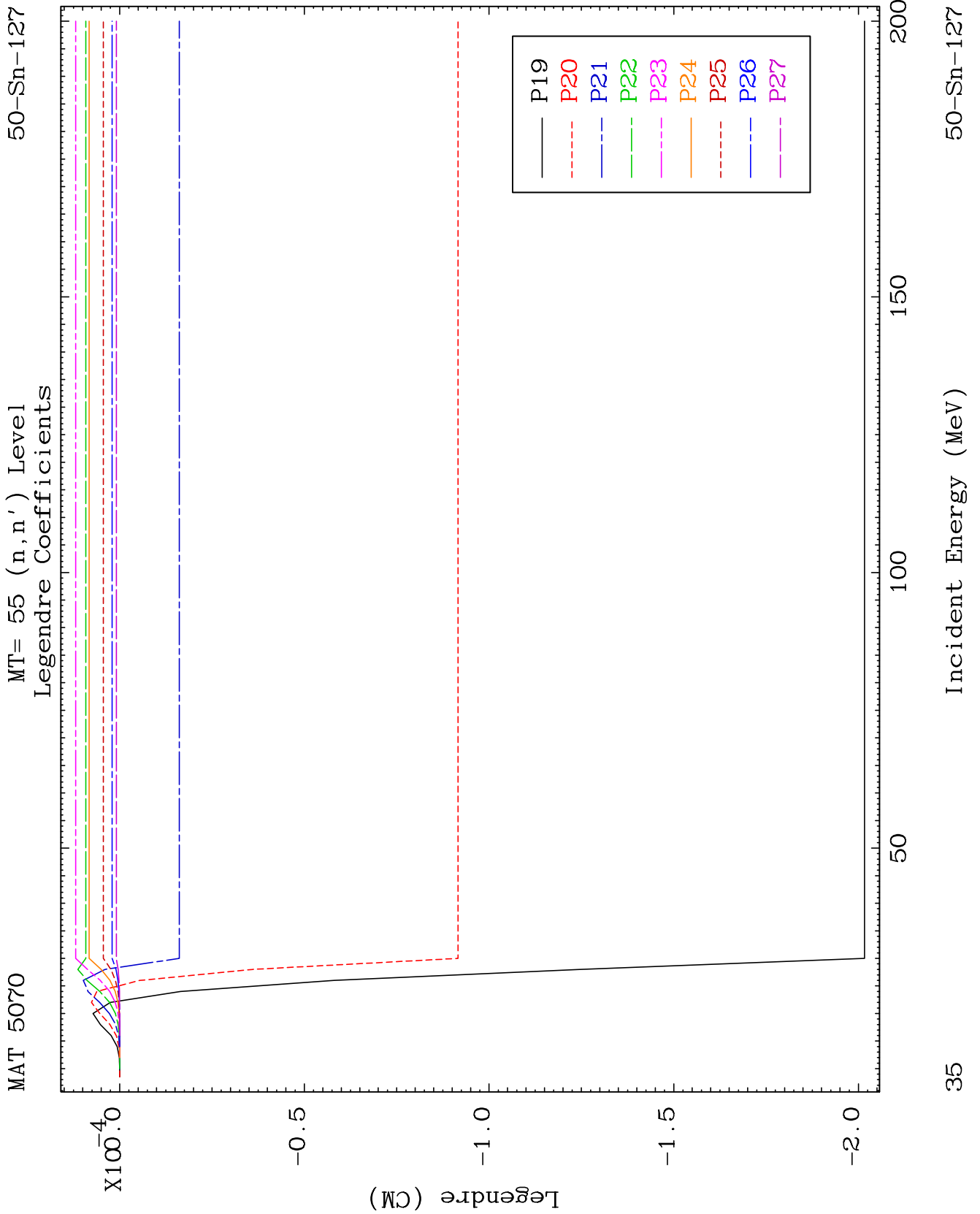


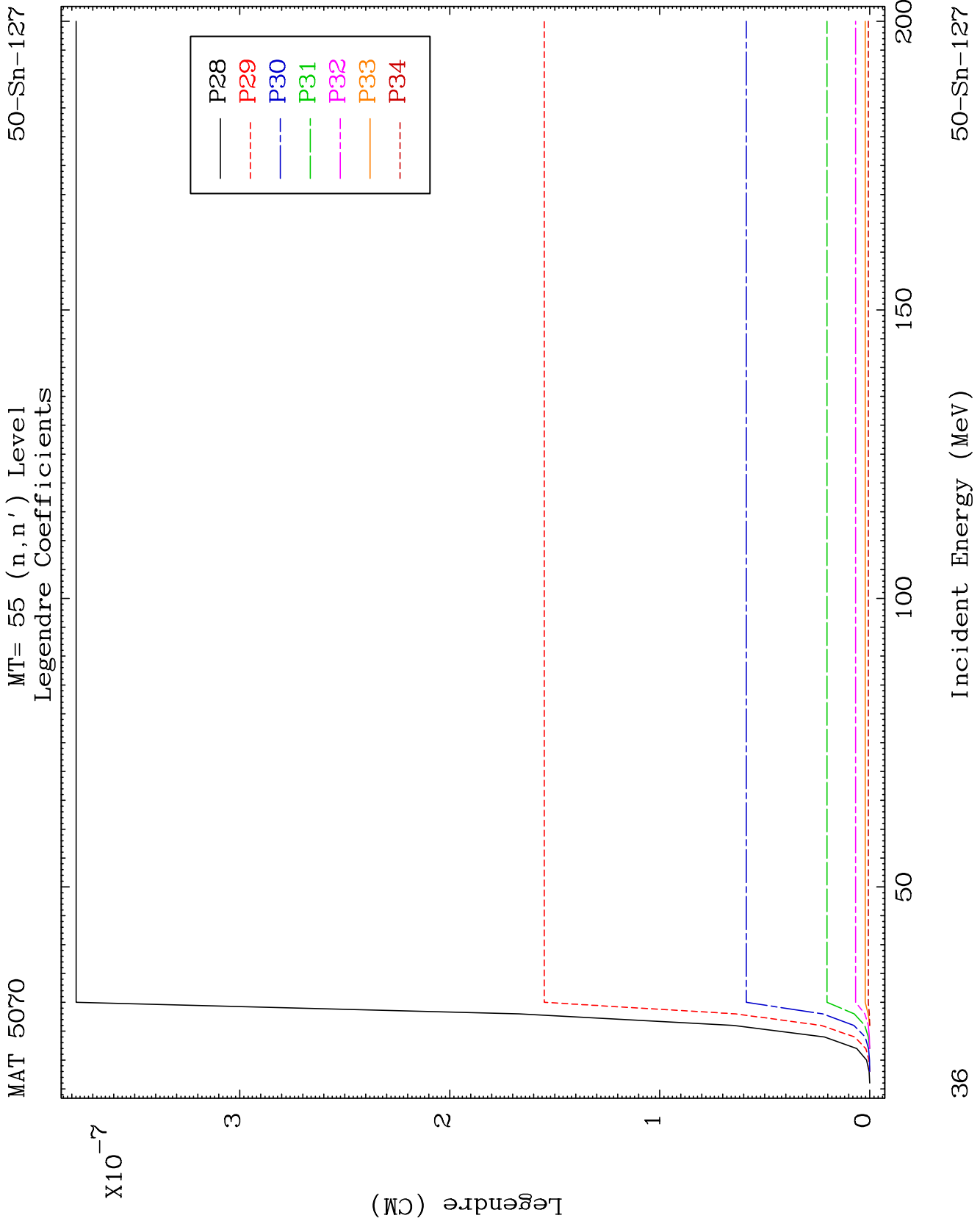


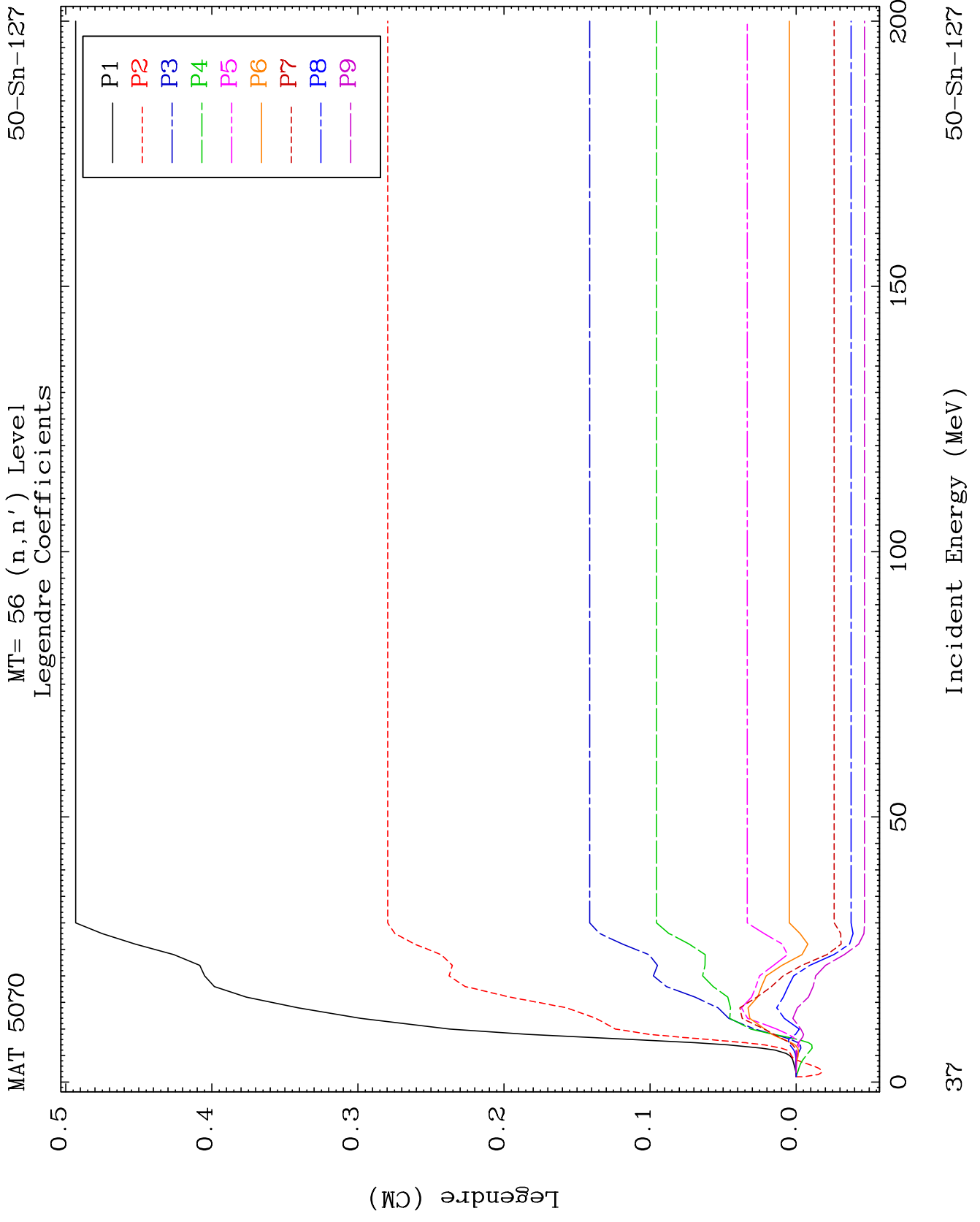


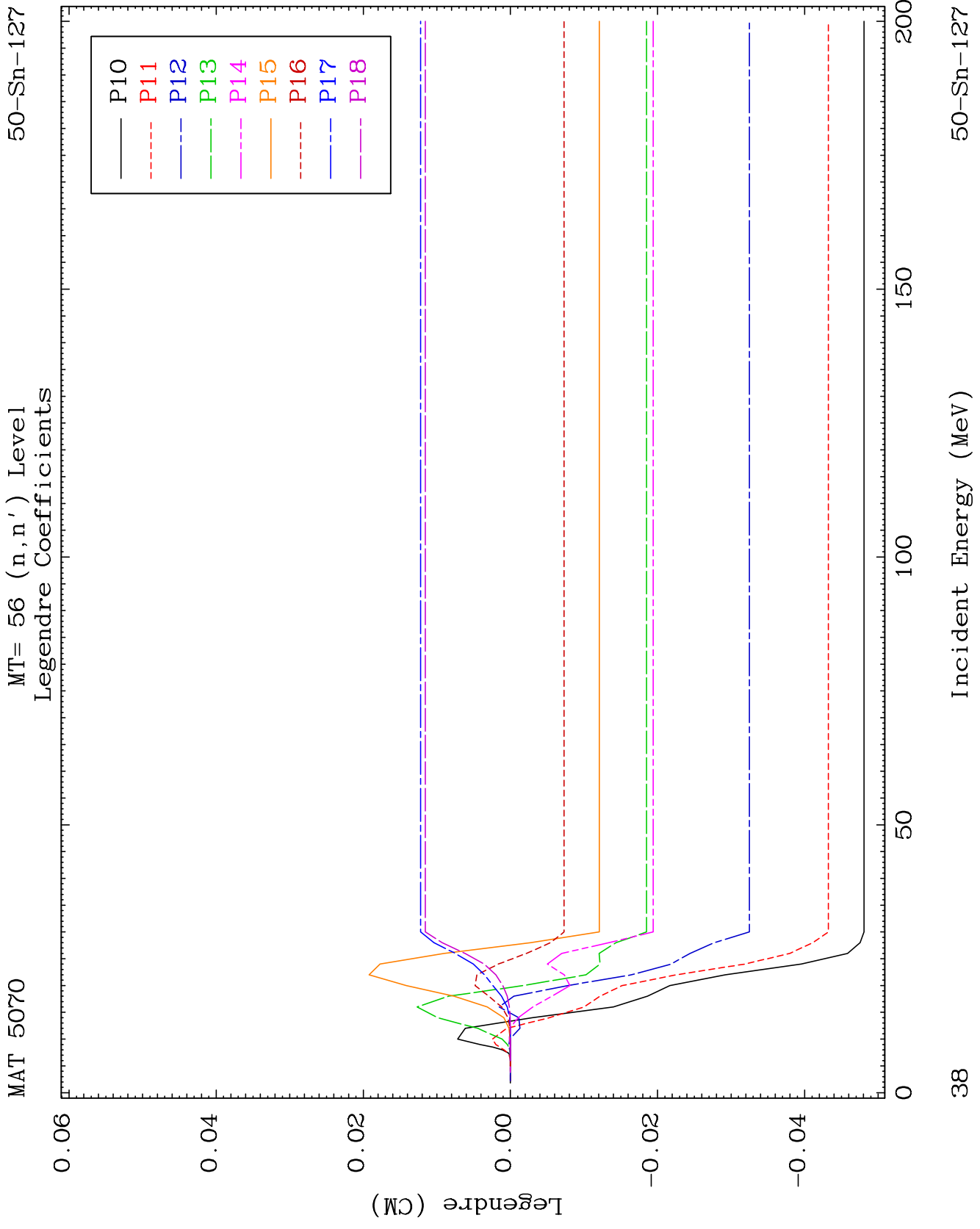


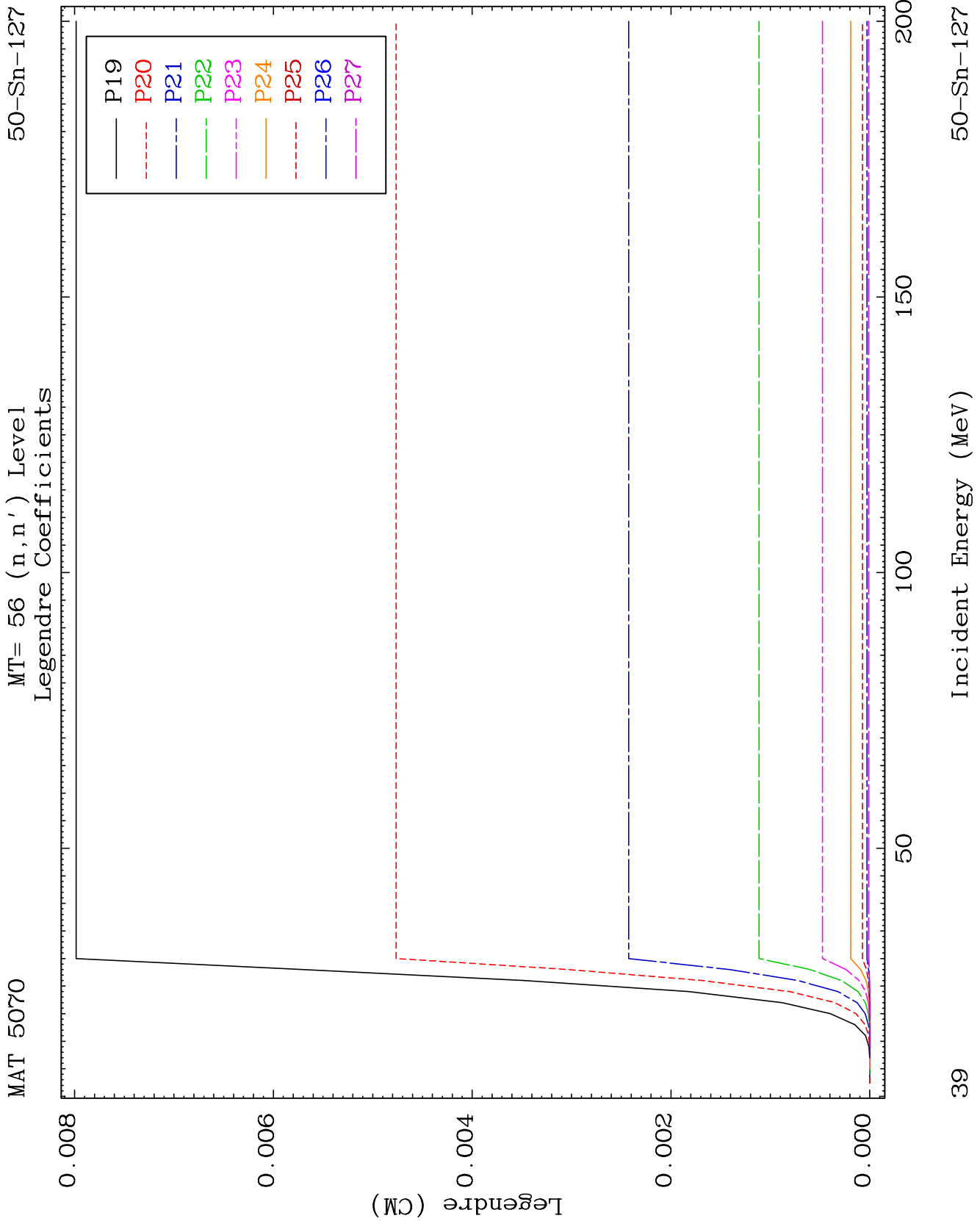




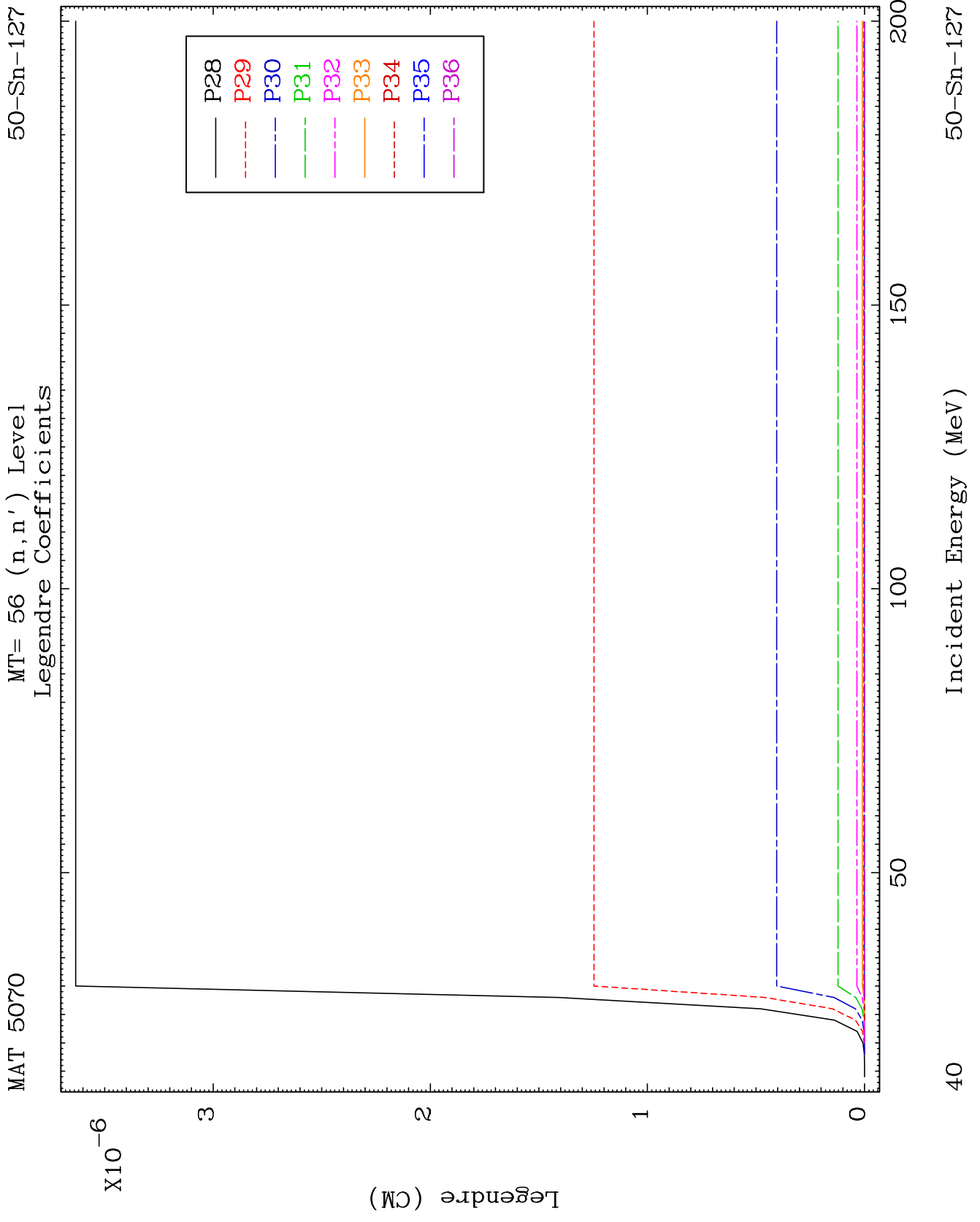










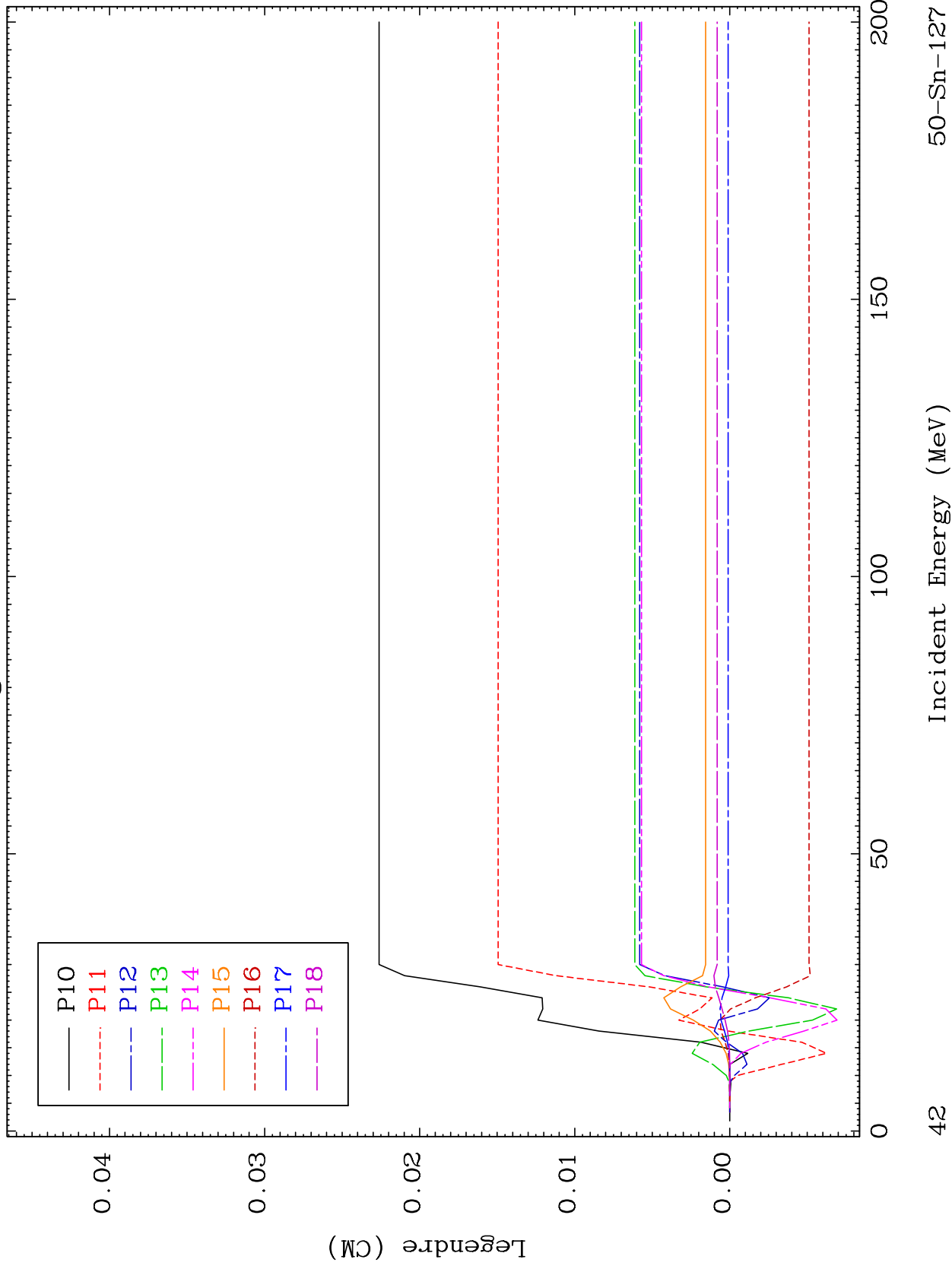




MAT 5070

MT= 57 (n,n') Level  
Legendre Coefficients

50-Sn-127



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Incident Energy (MeV)

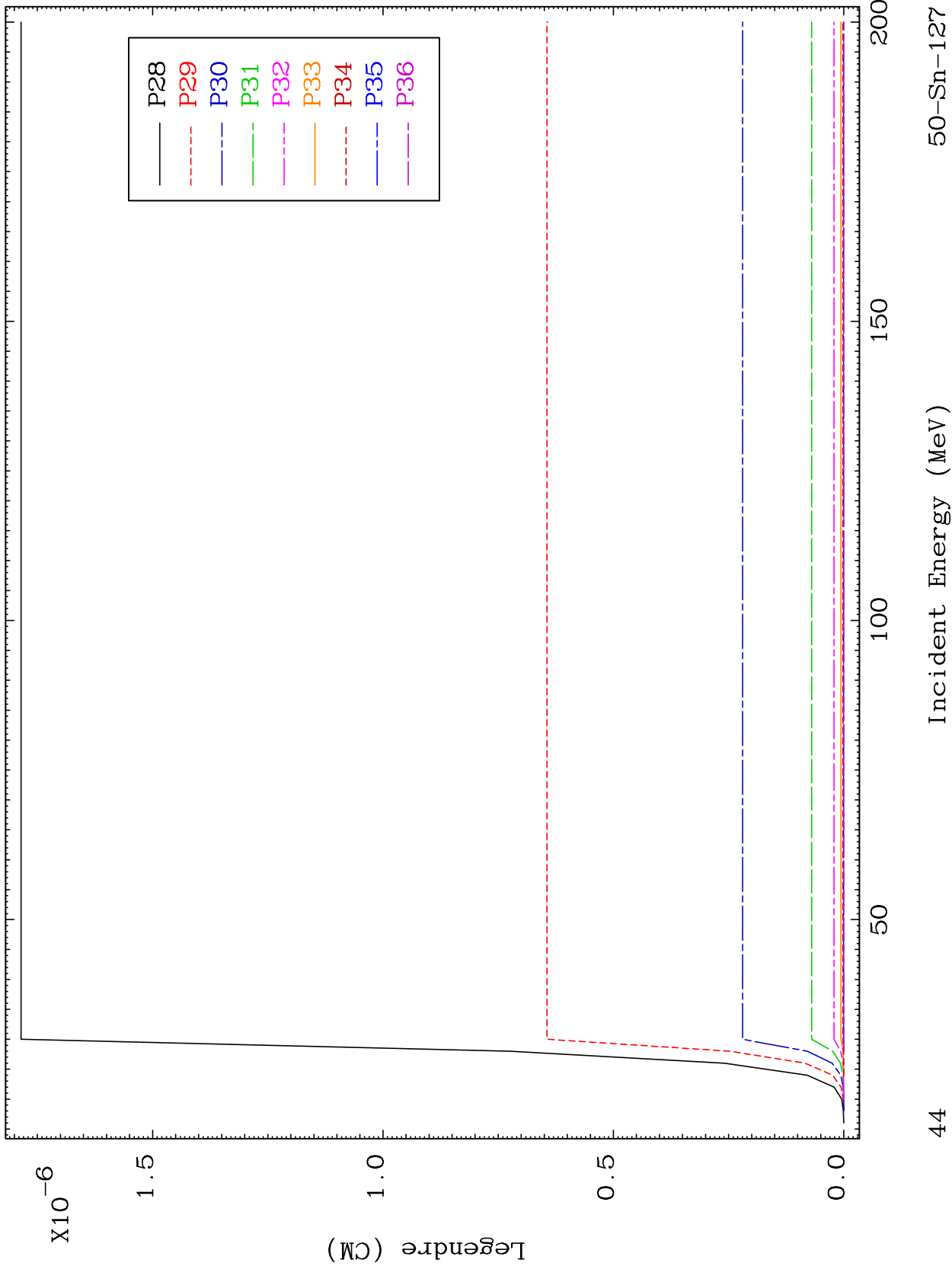
50-Sn-127



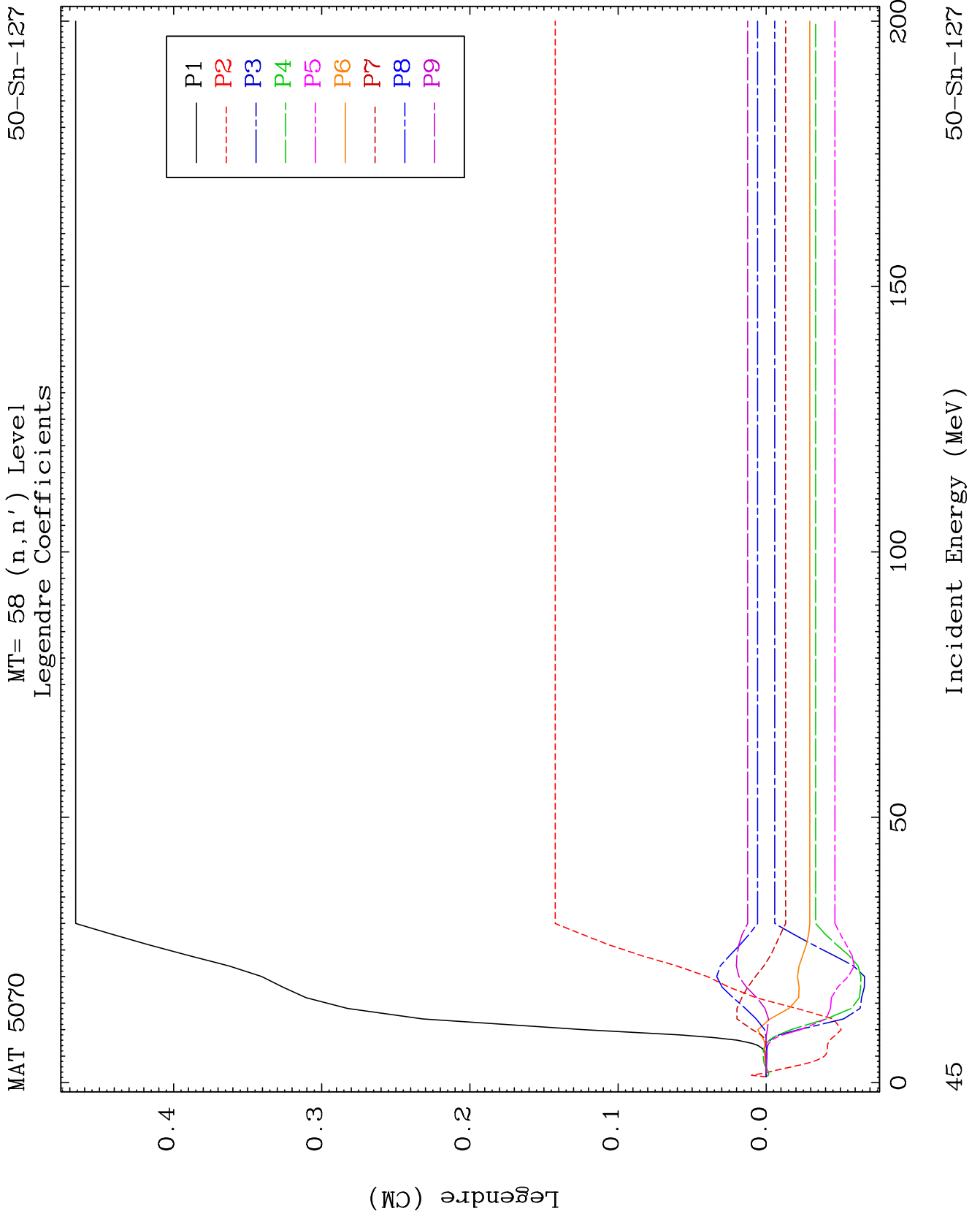
MAT 5070

MT= 57 (n,n') Level  
Legendre Coefficients

50-Sn-127



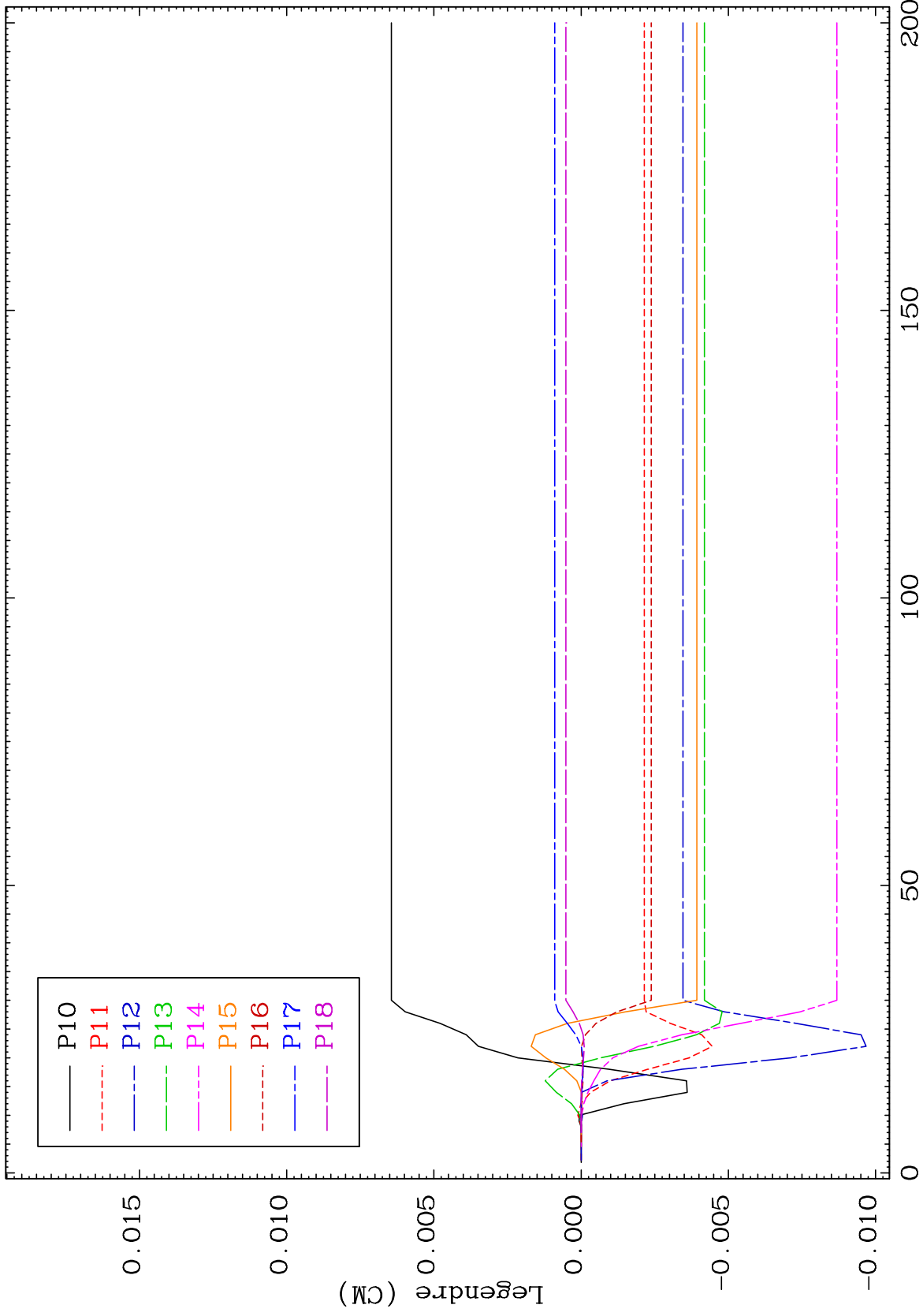
44



MAT 5070

MT= 58 (n,n') Level  
Legendre Coefficients

50-Sn-127



46

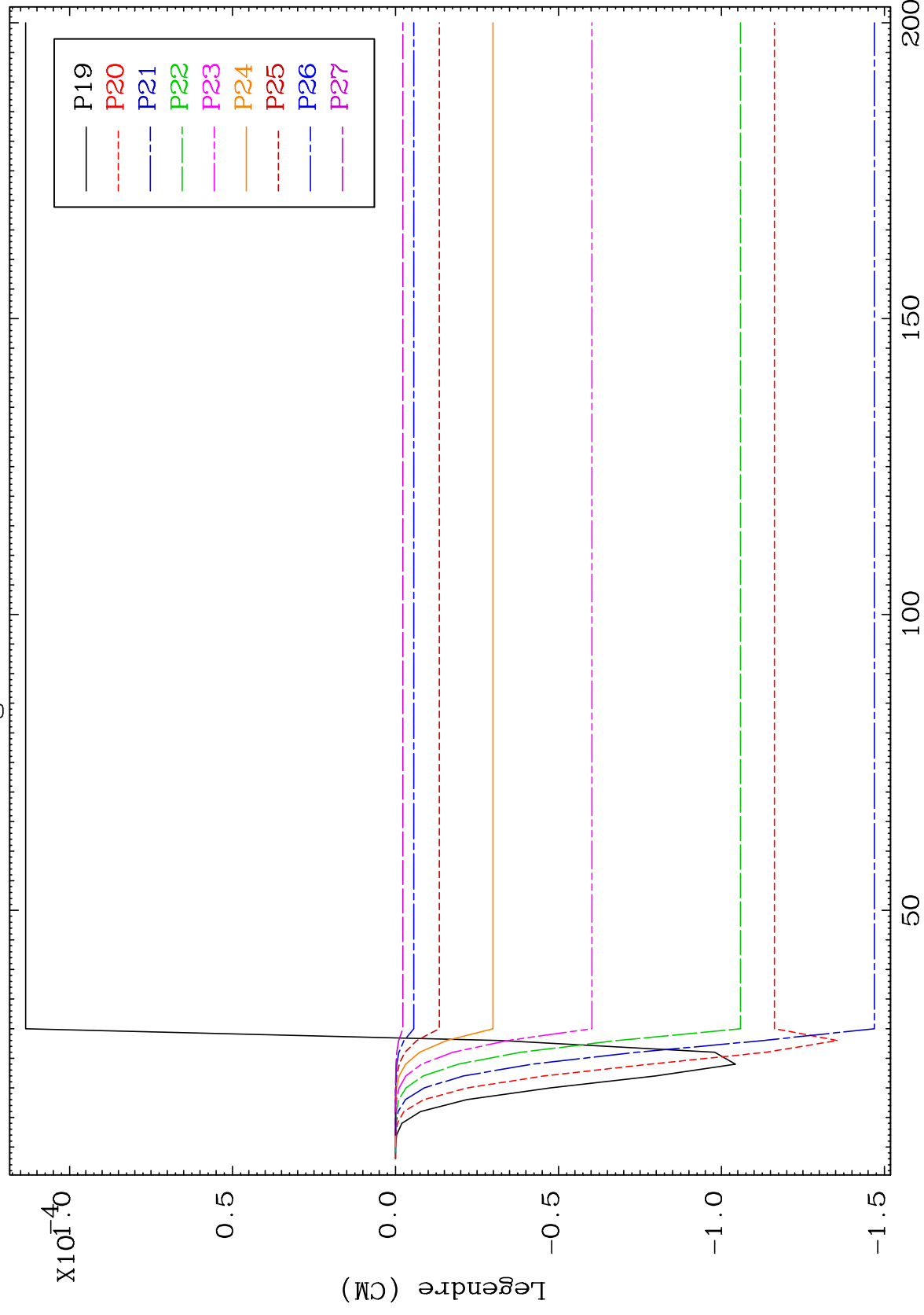
Incident Energy (MeV)

50-Sn-127

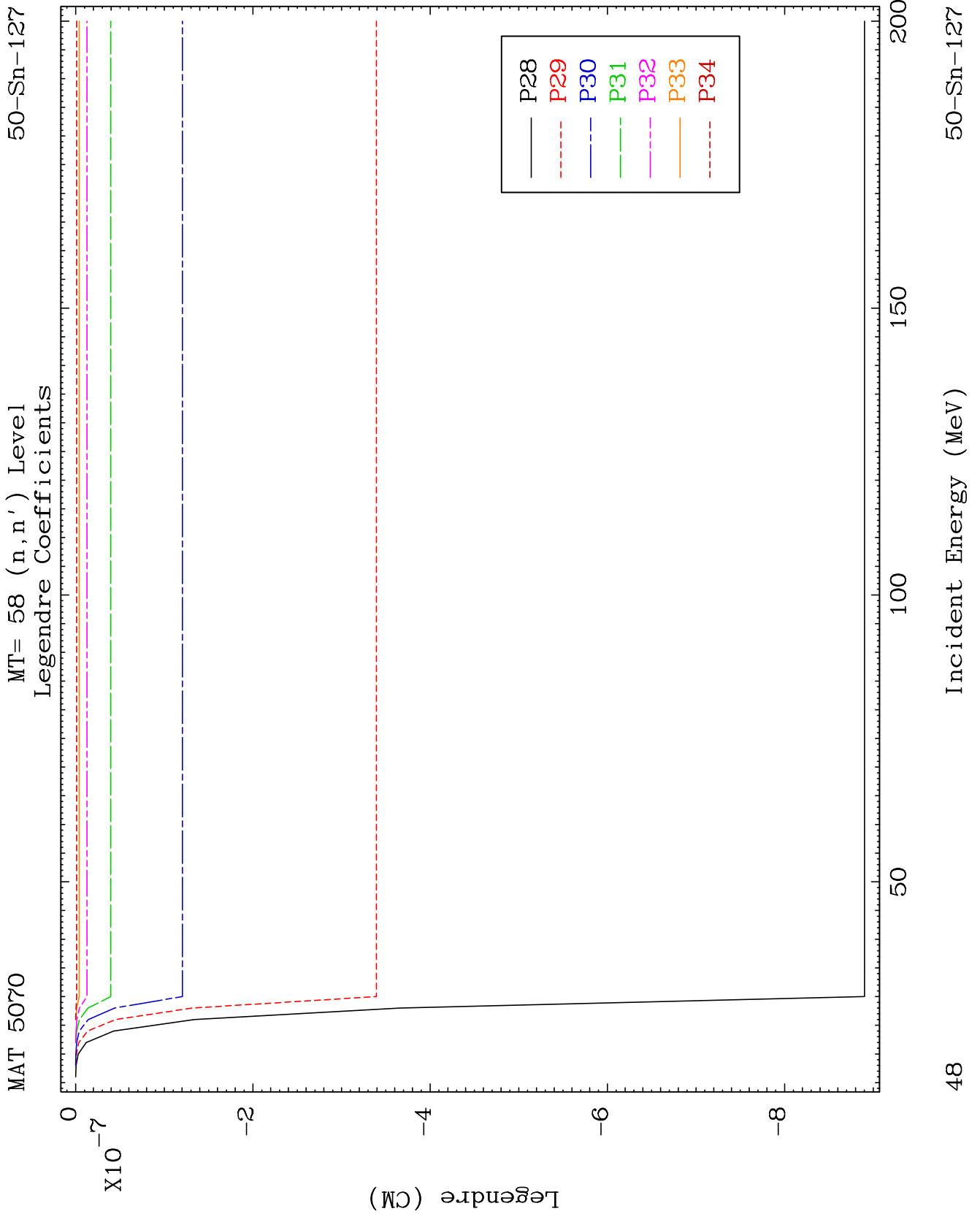
MAT 5070

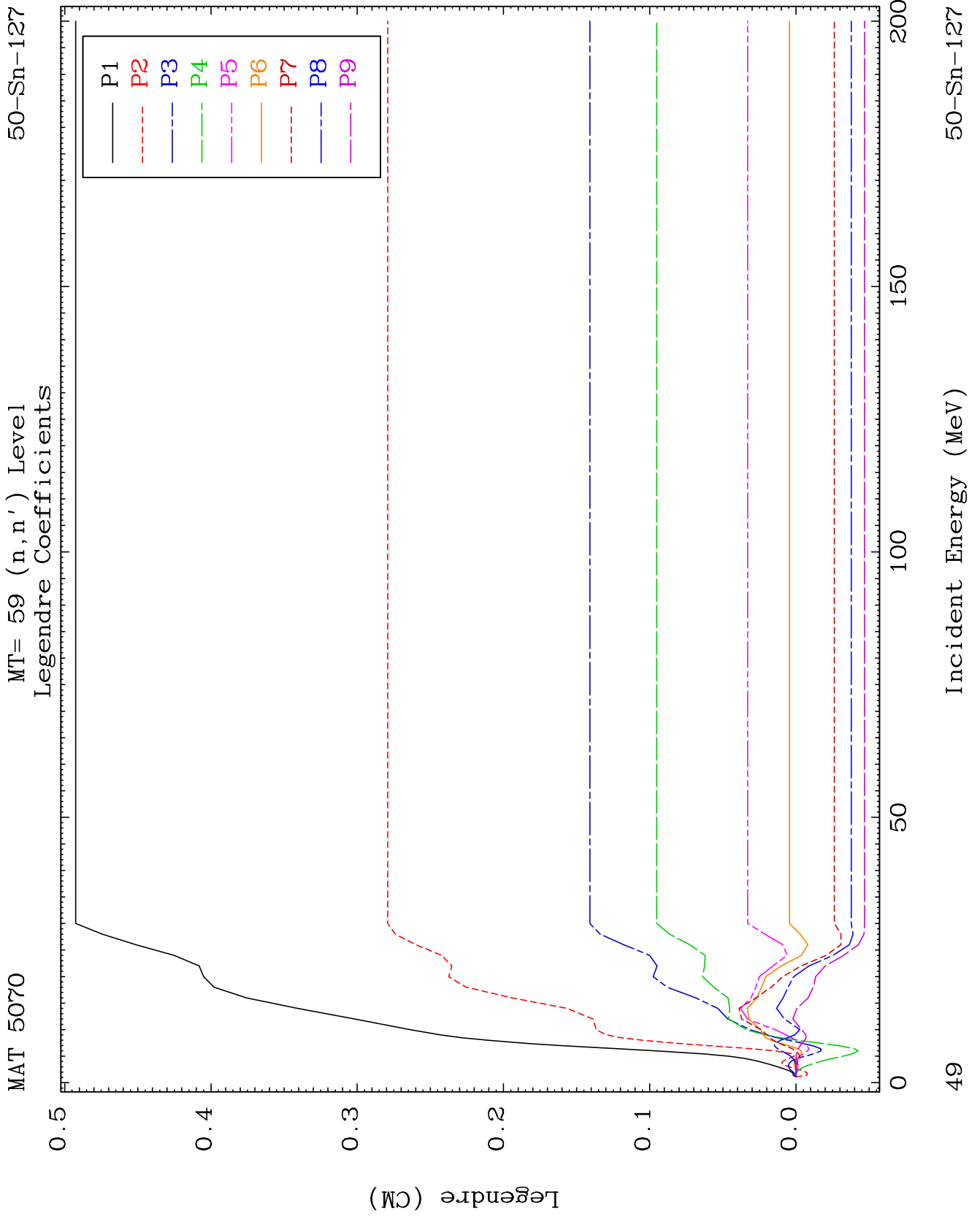
MT= 58 (n,n') Level  
Legendre Coefficients

50-Sn-127

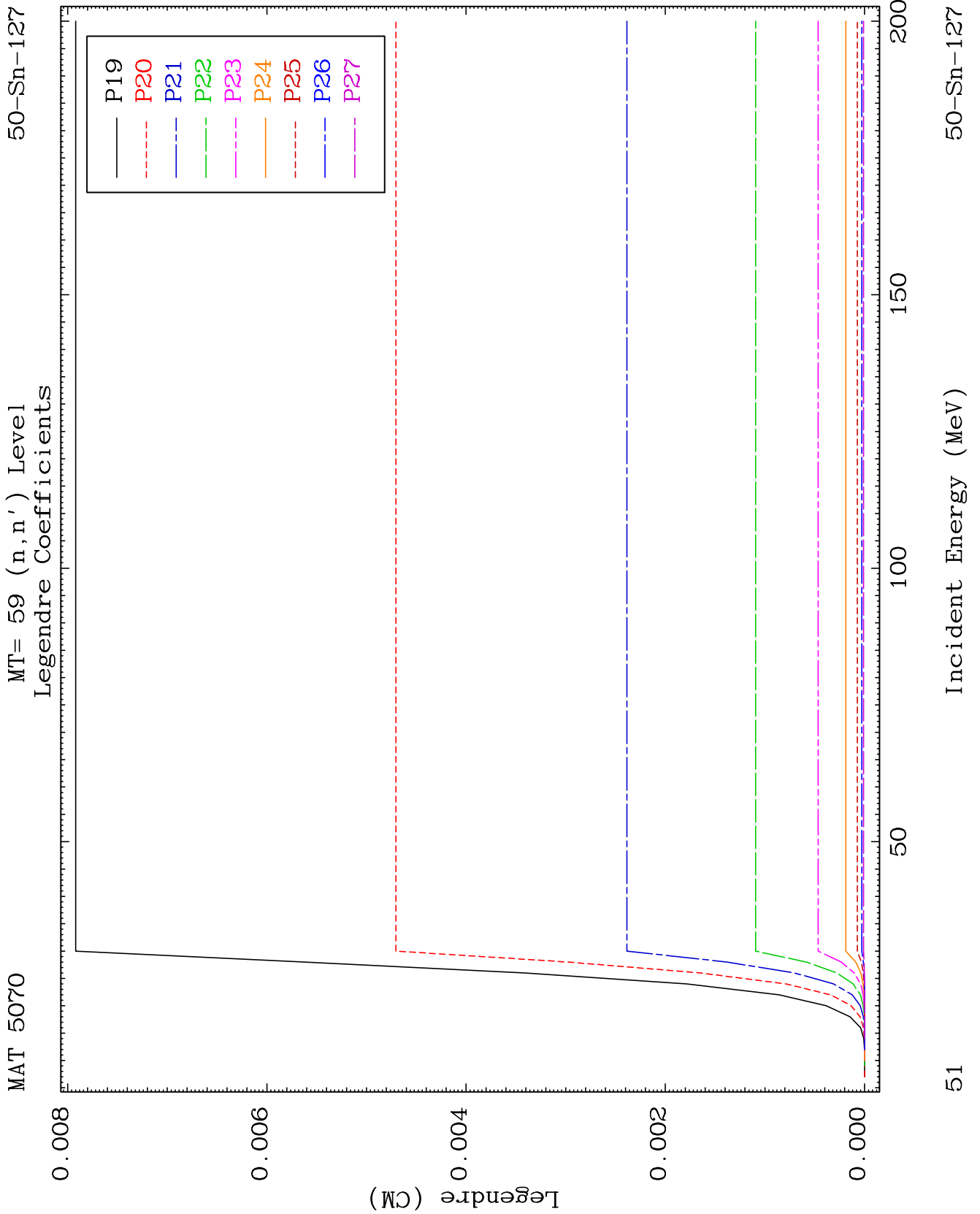


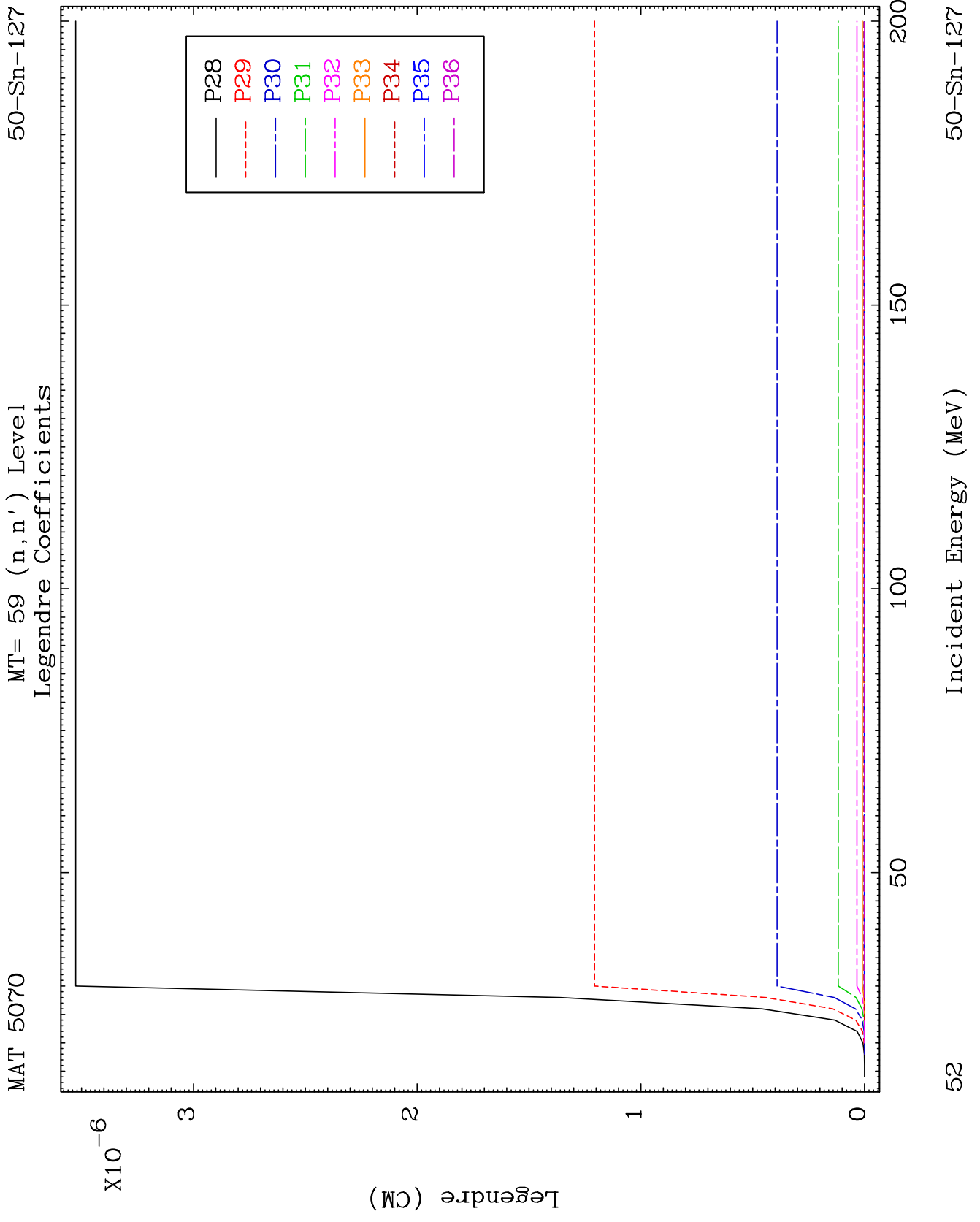




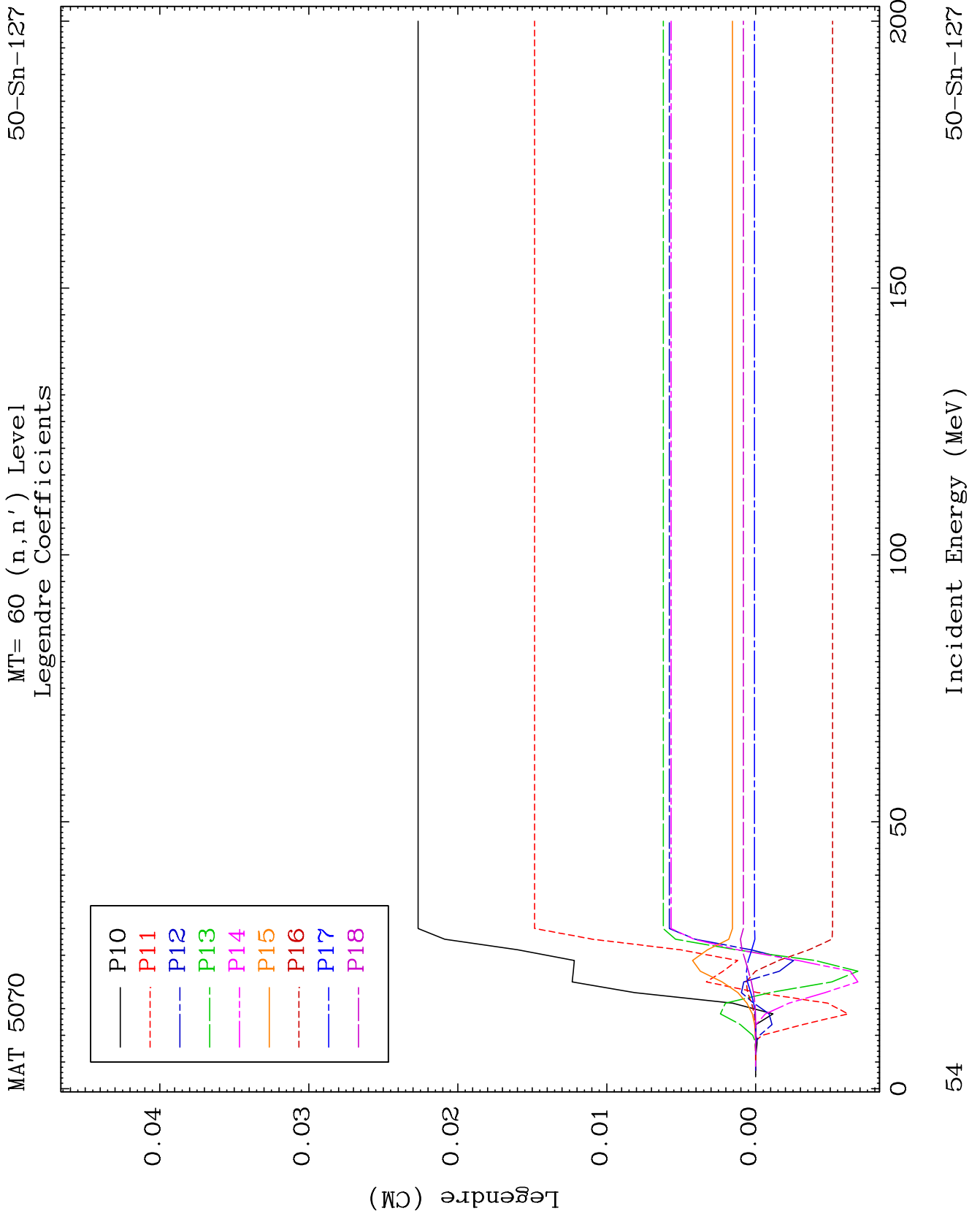


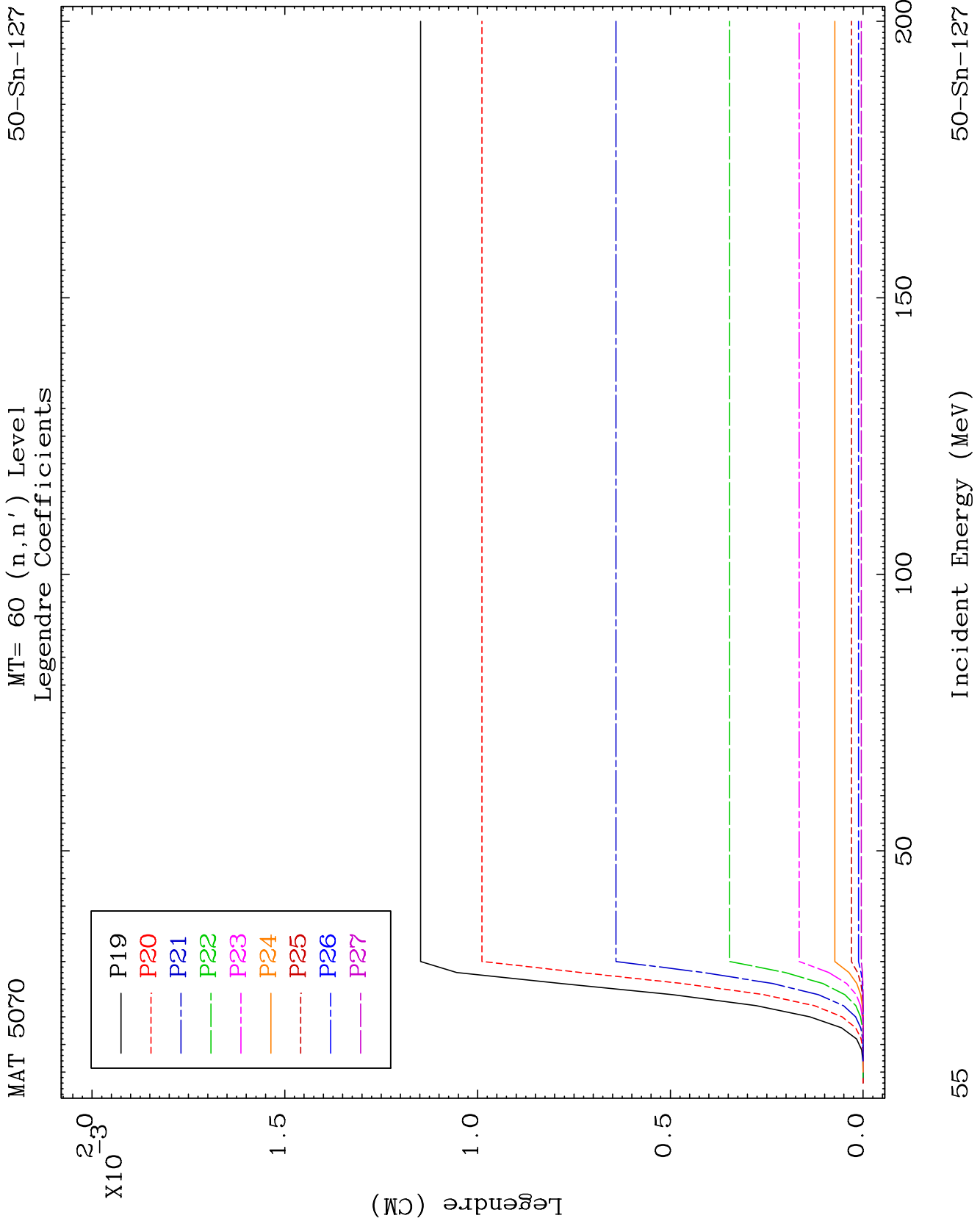




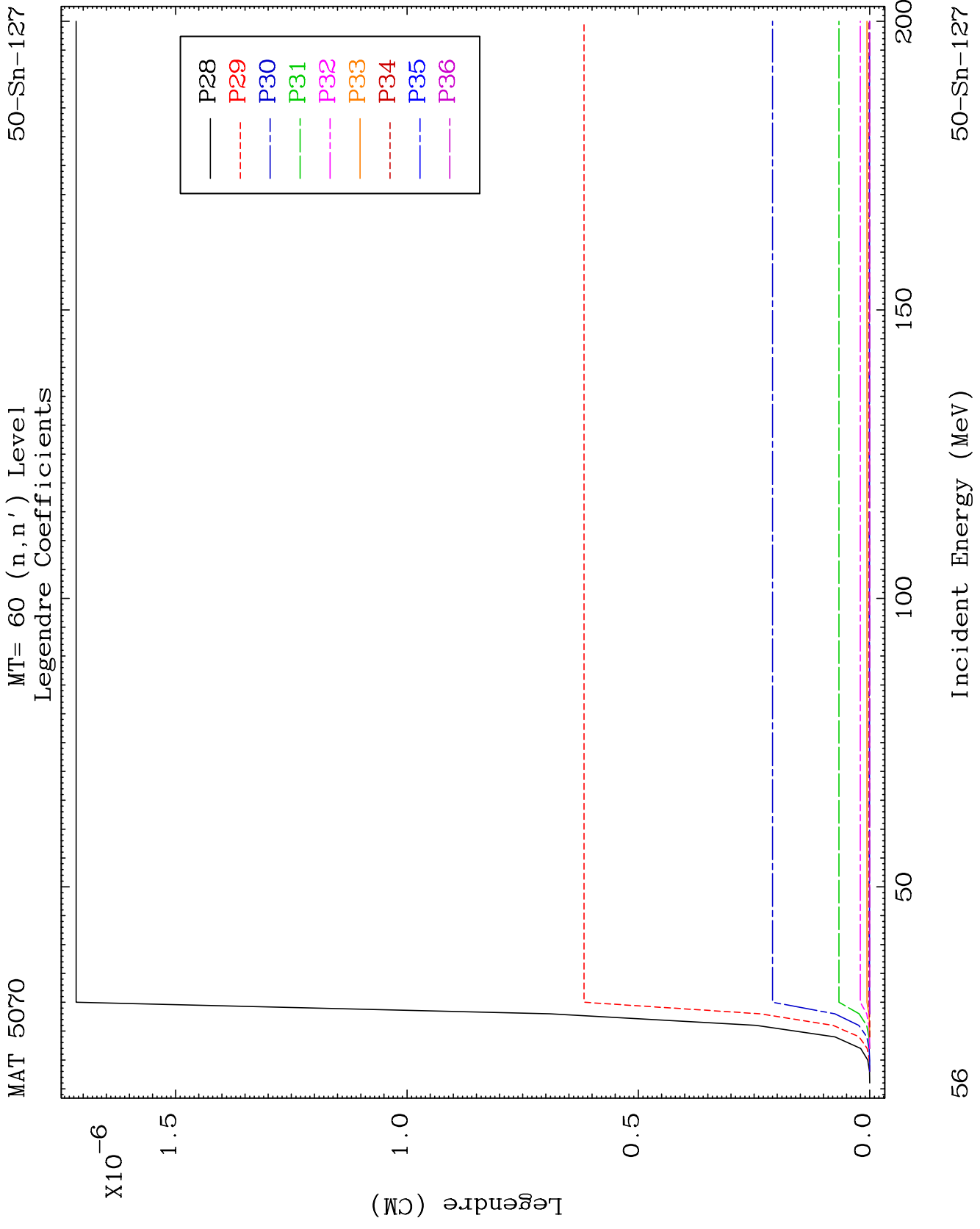


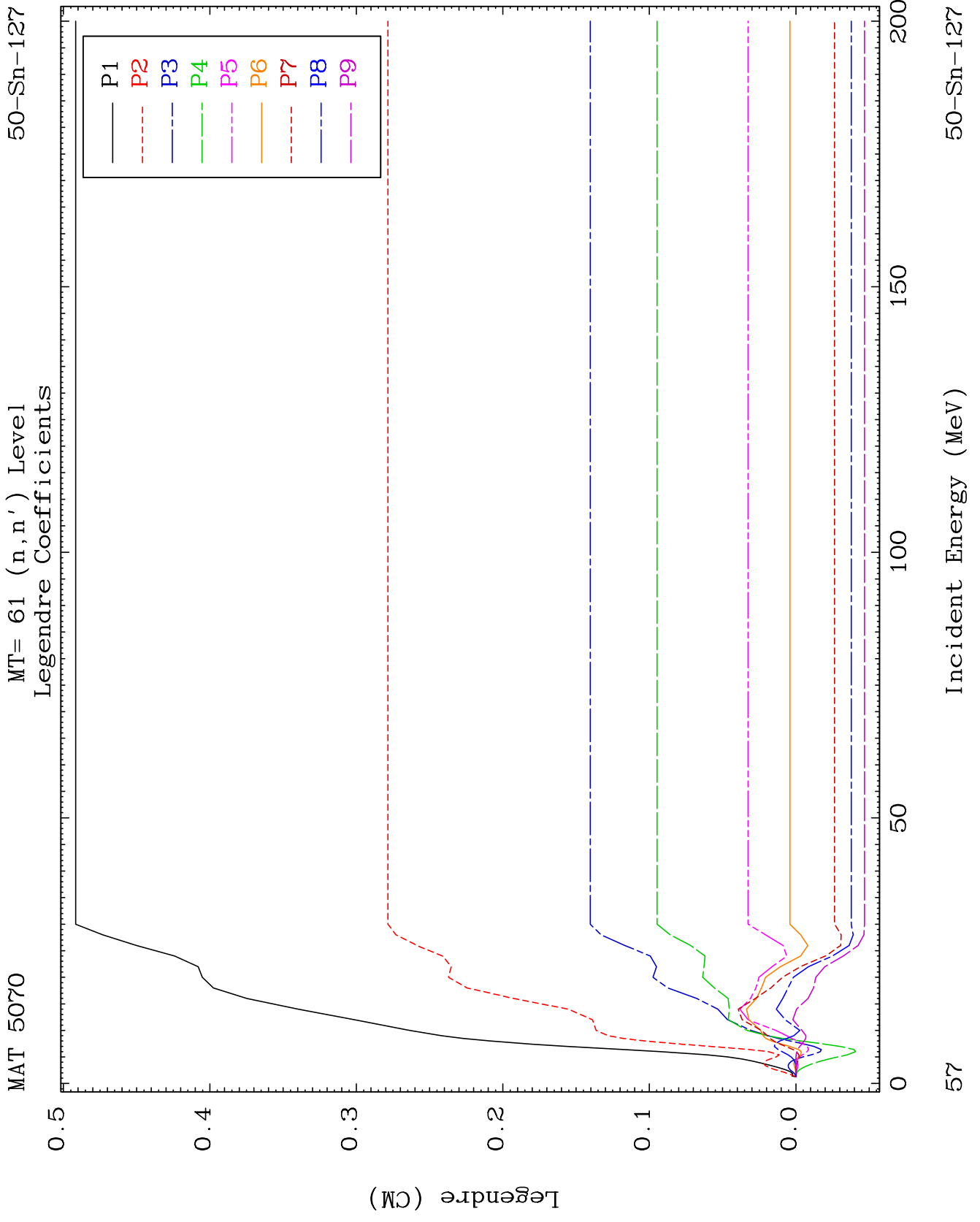


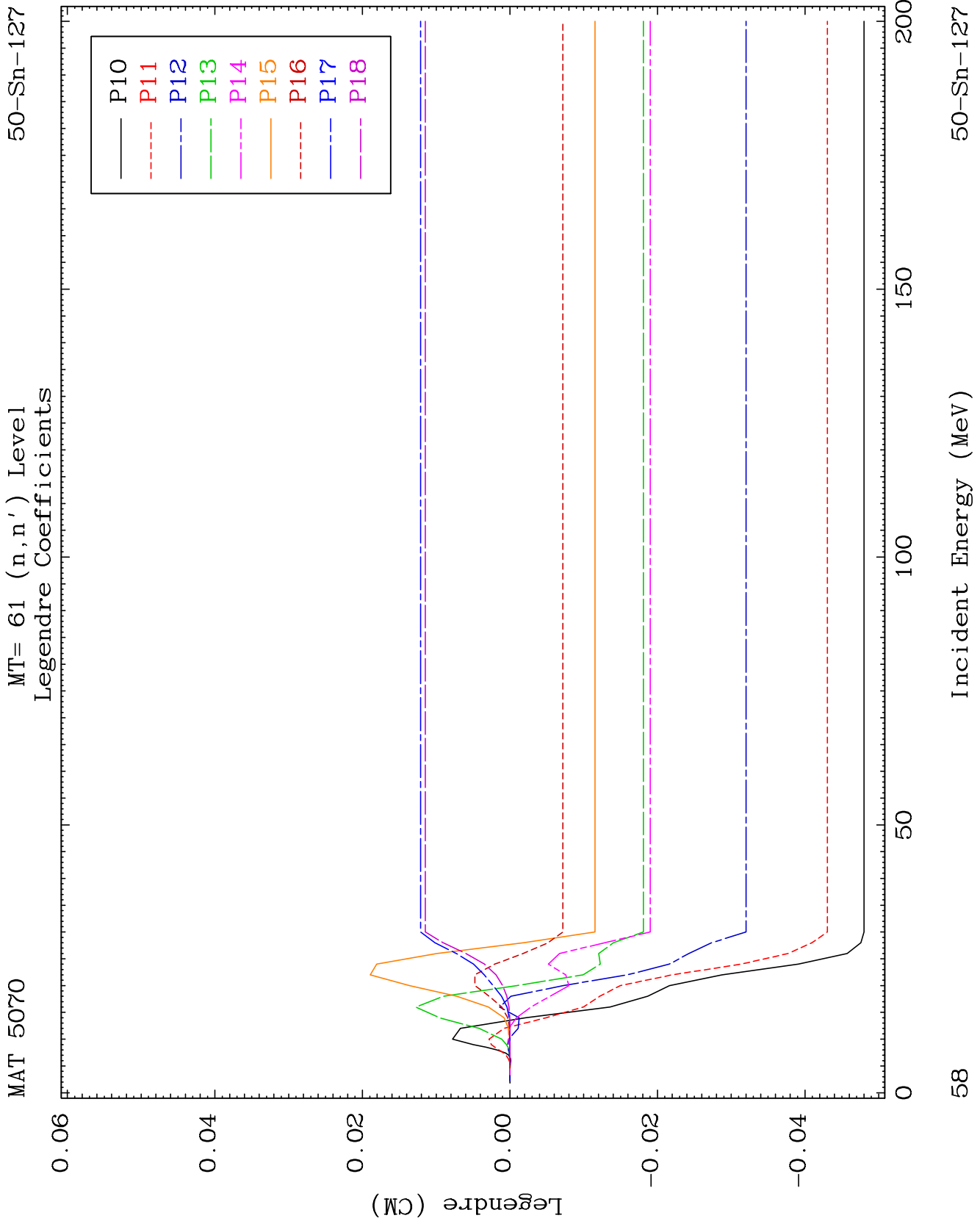


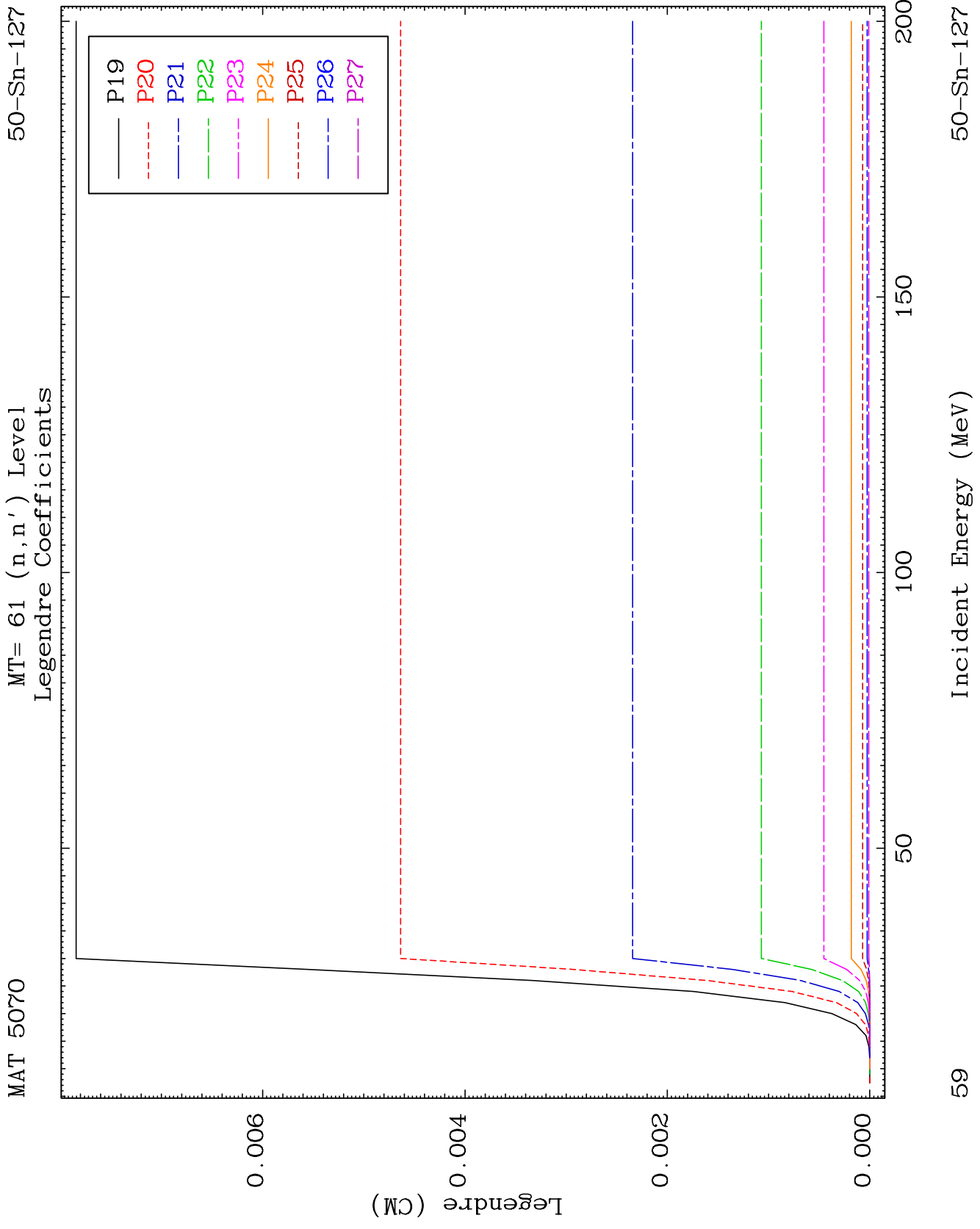


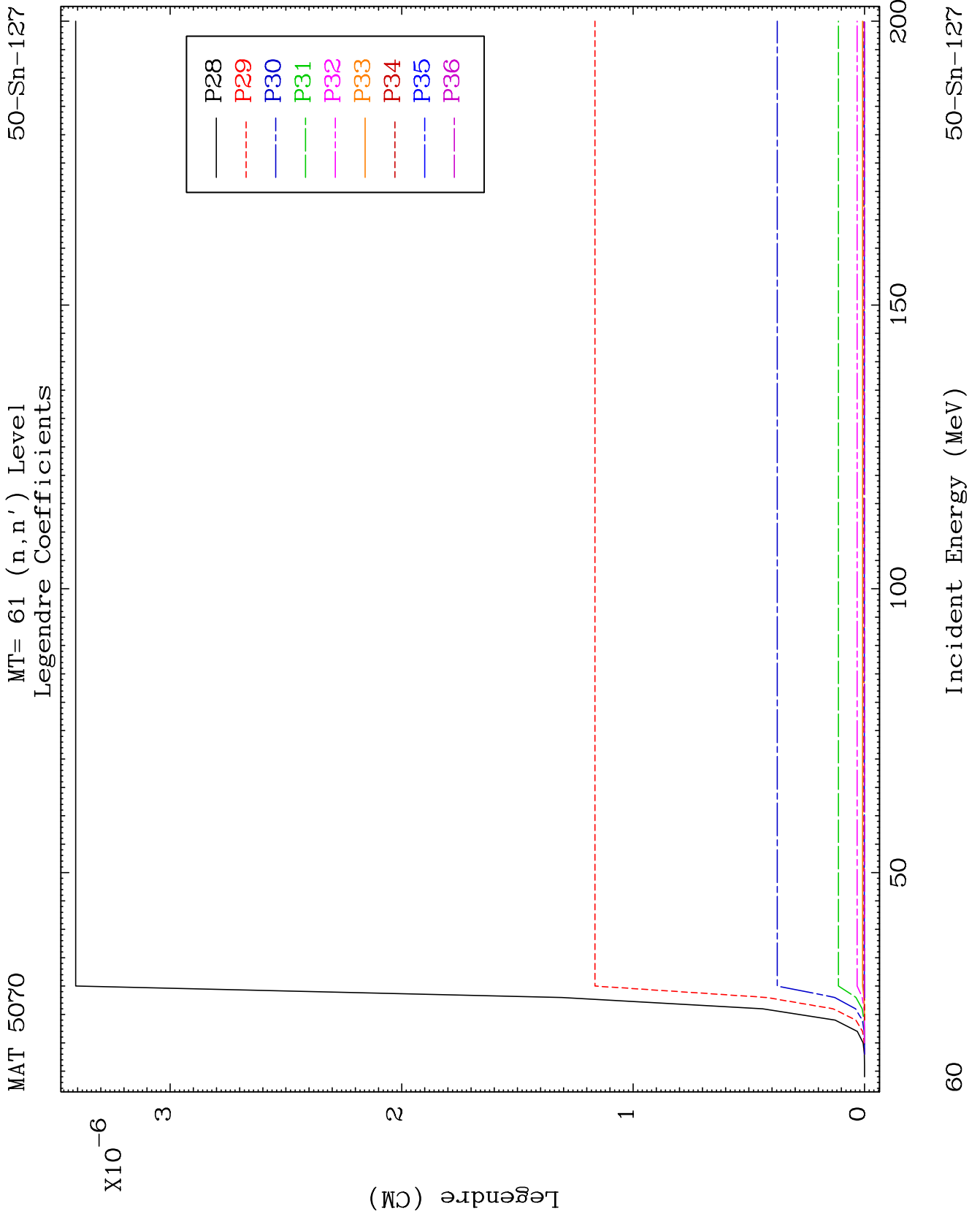


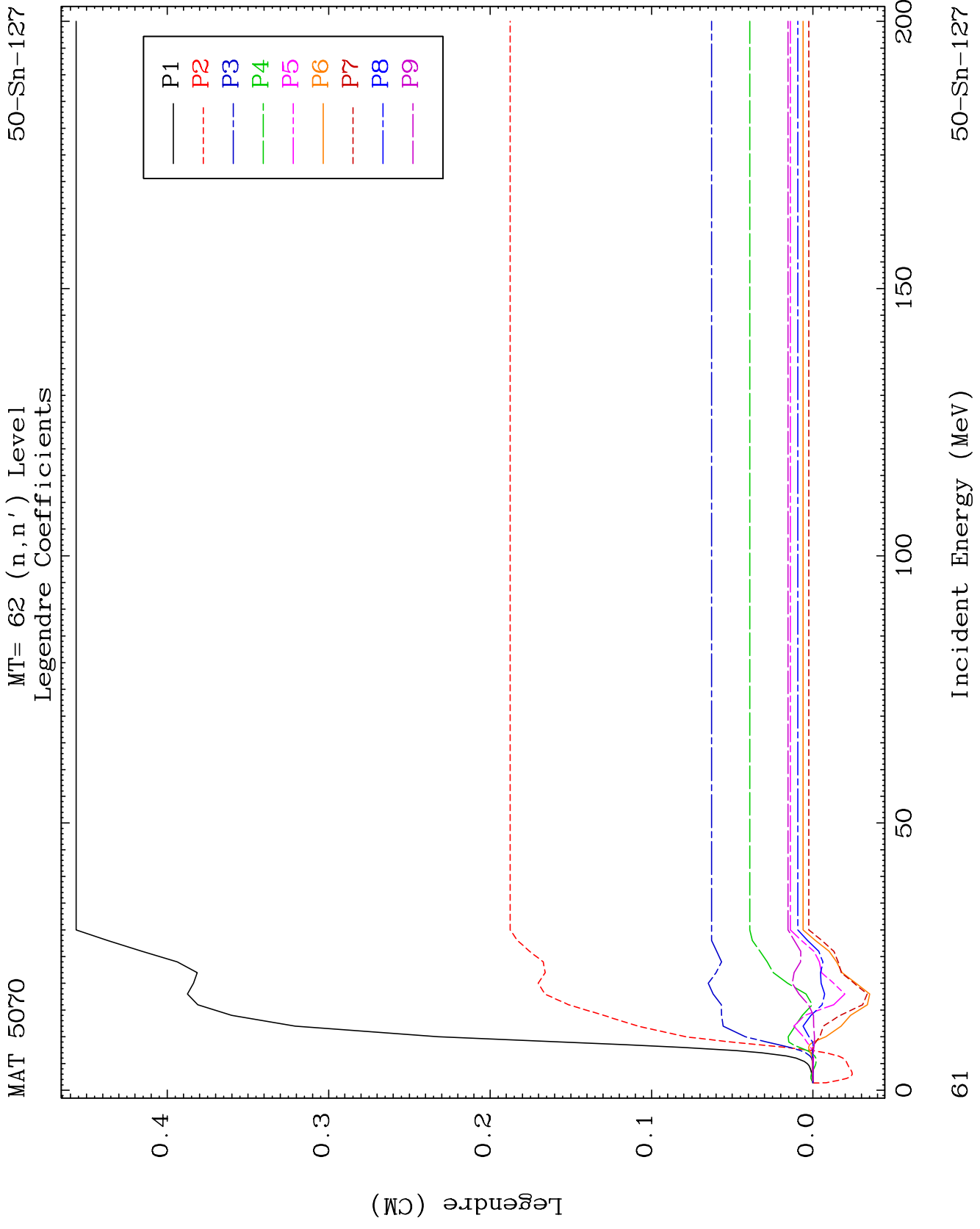










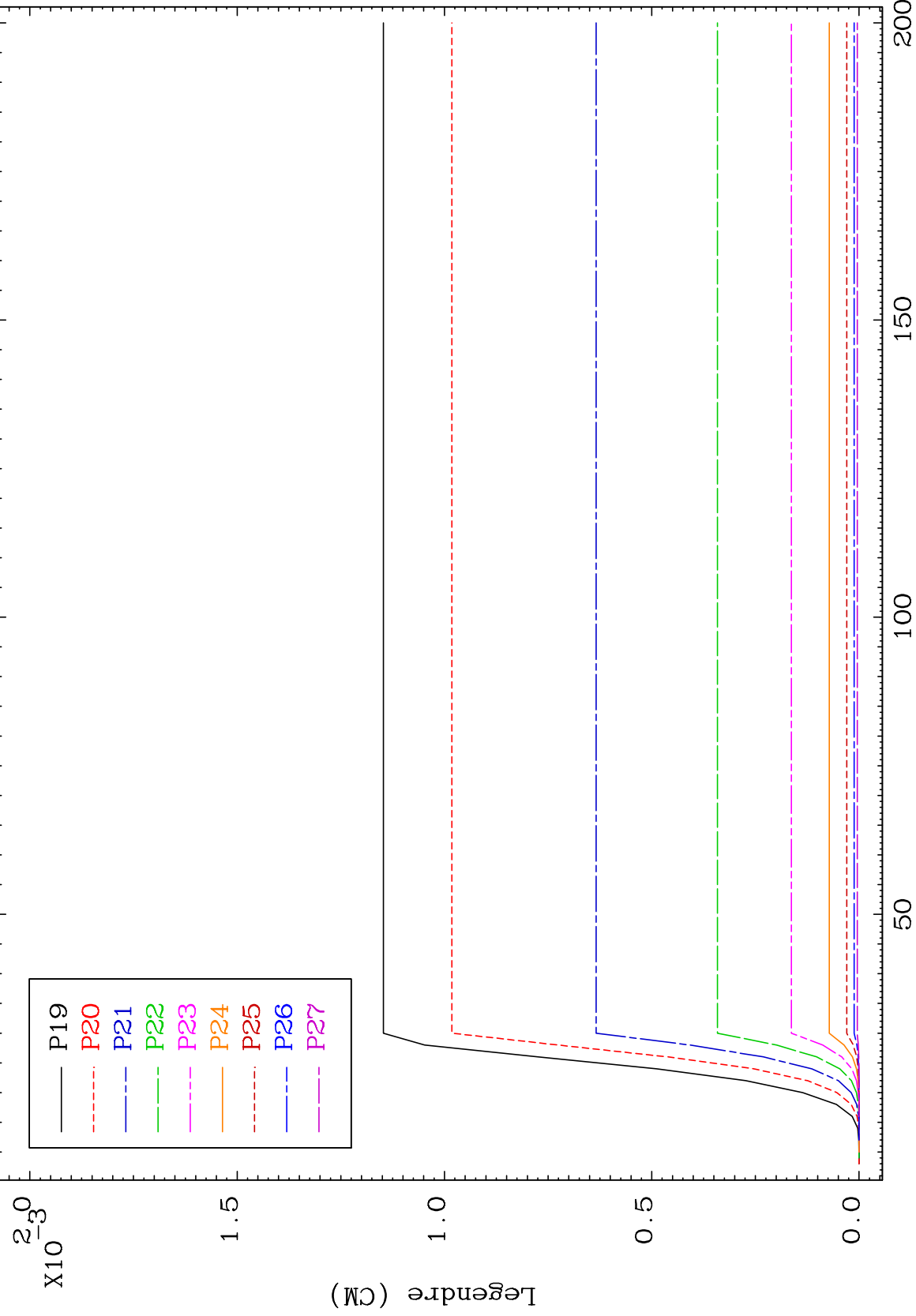




MAT 5070

MT= 62 (n,n') Level  
Legendre Coefficients

50-Sn-127

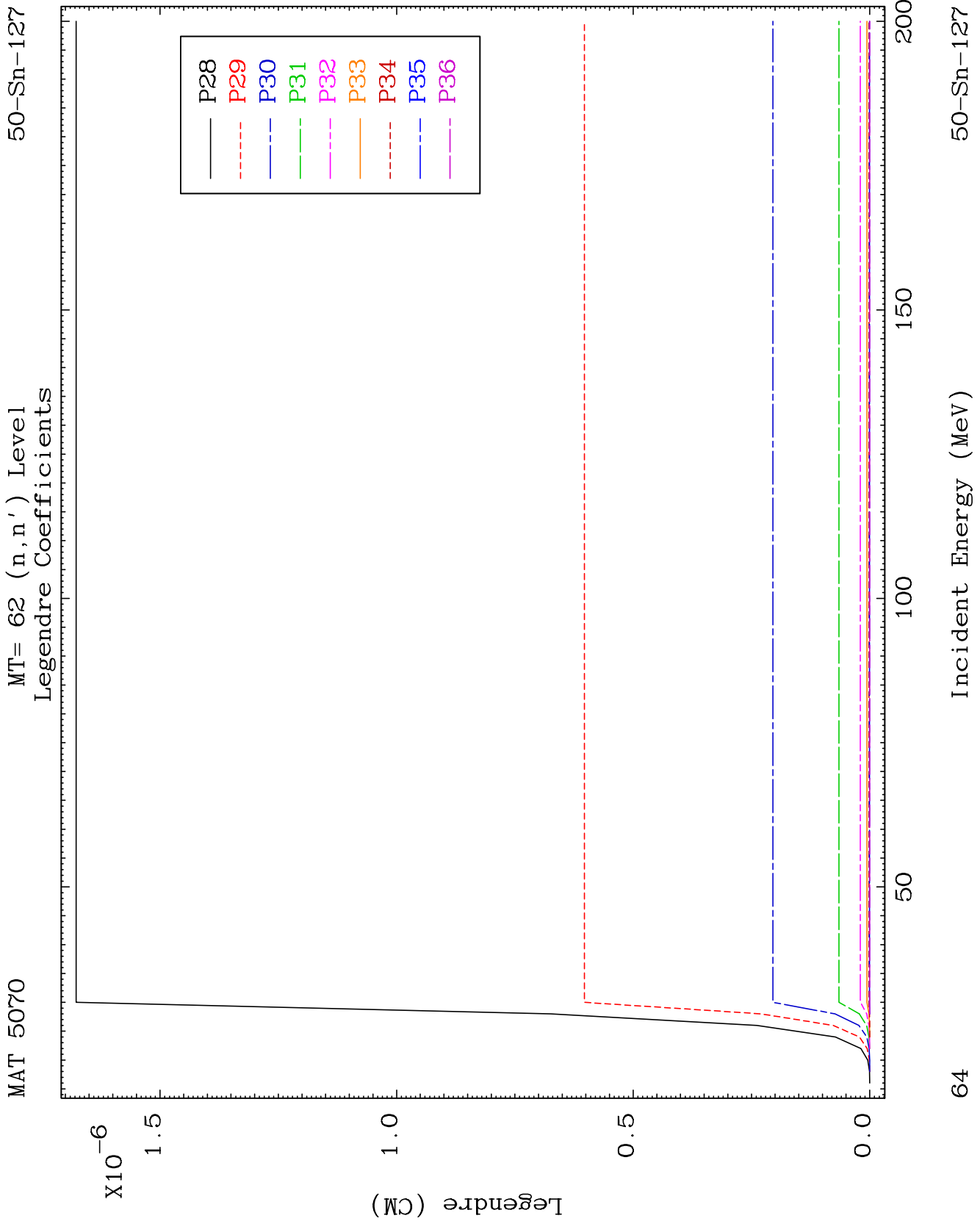


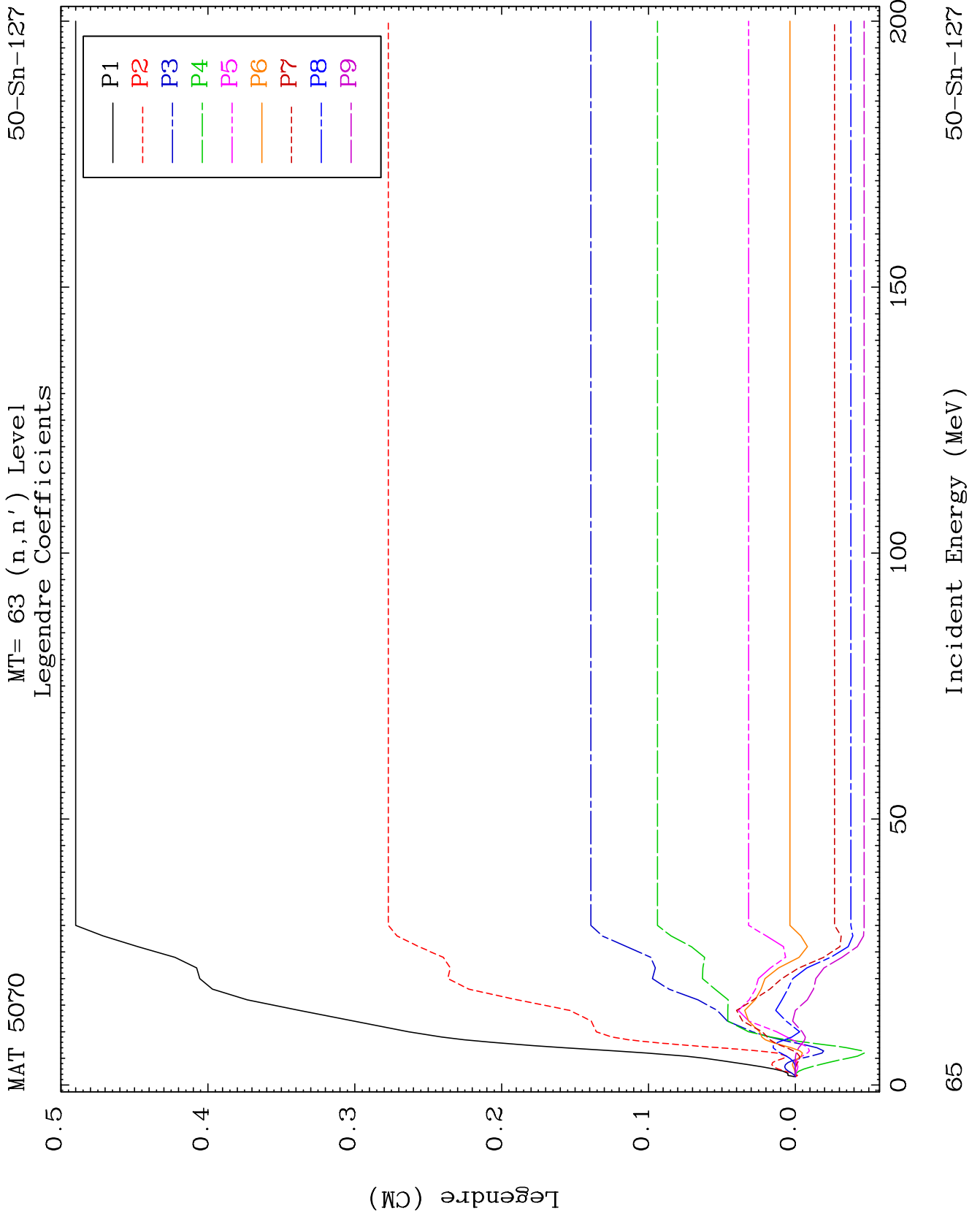
63

Incident Energy (MeV)

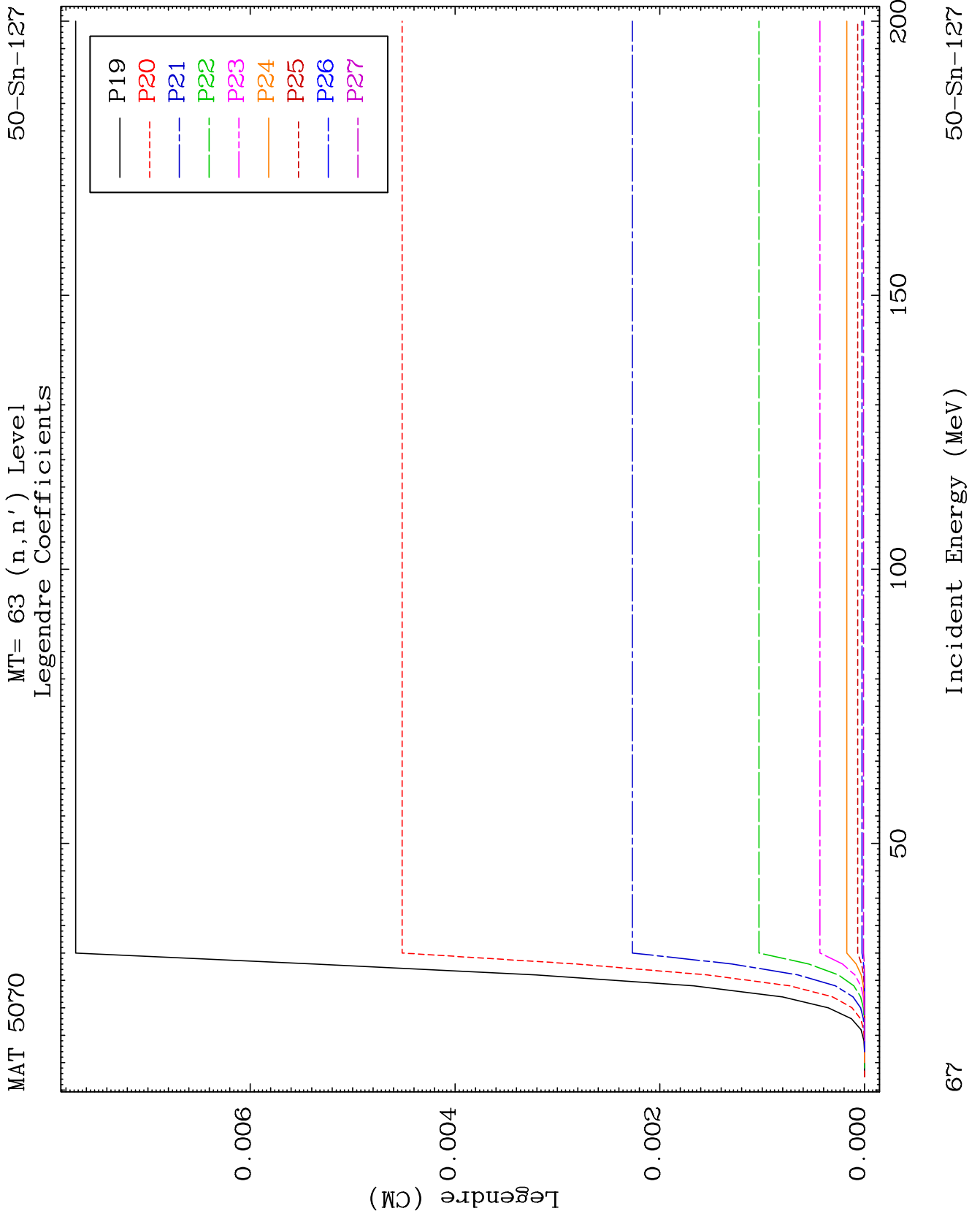
50-Sn-127

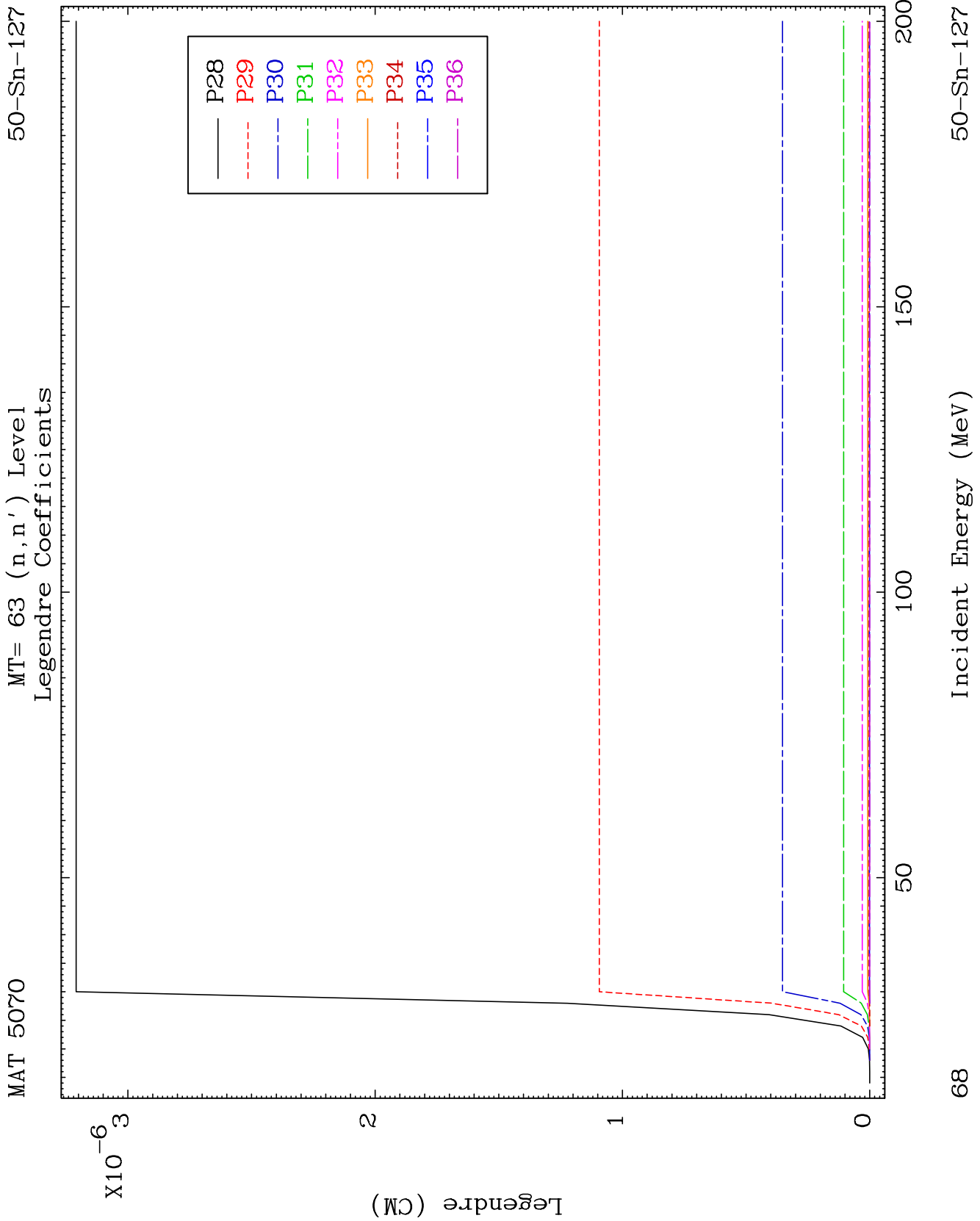


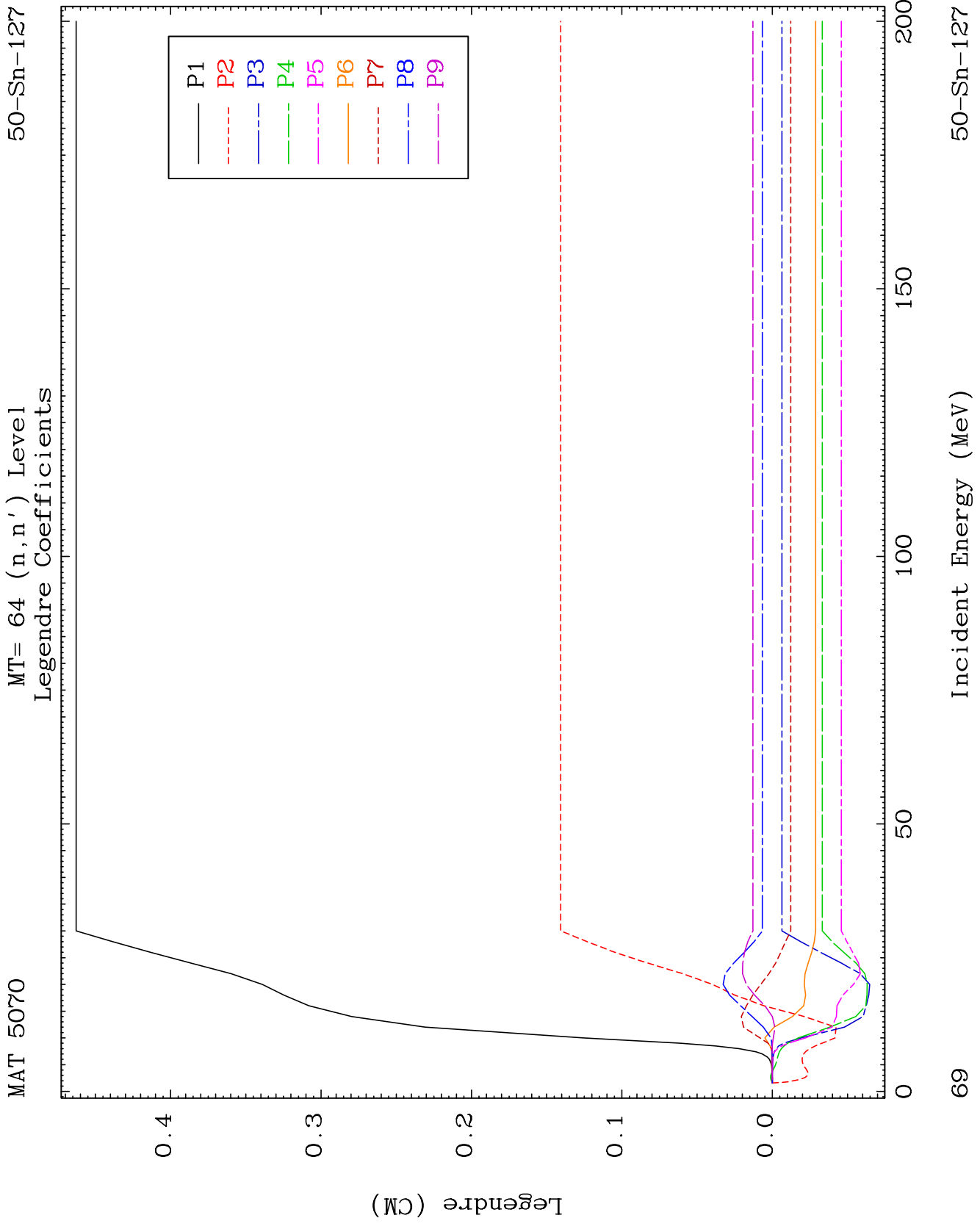








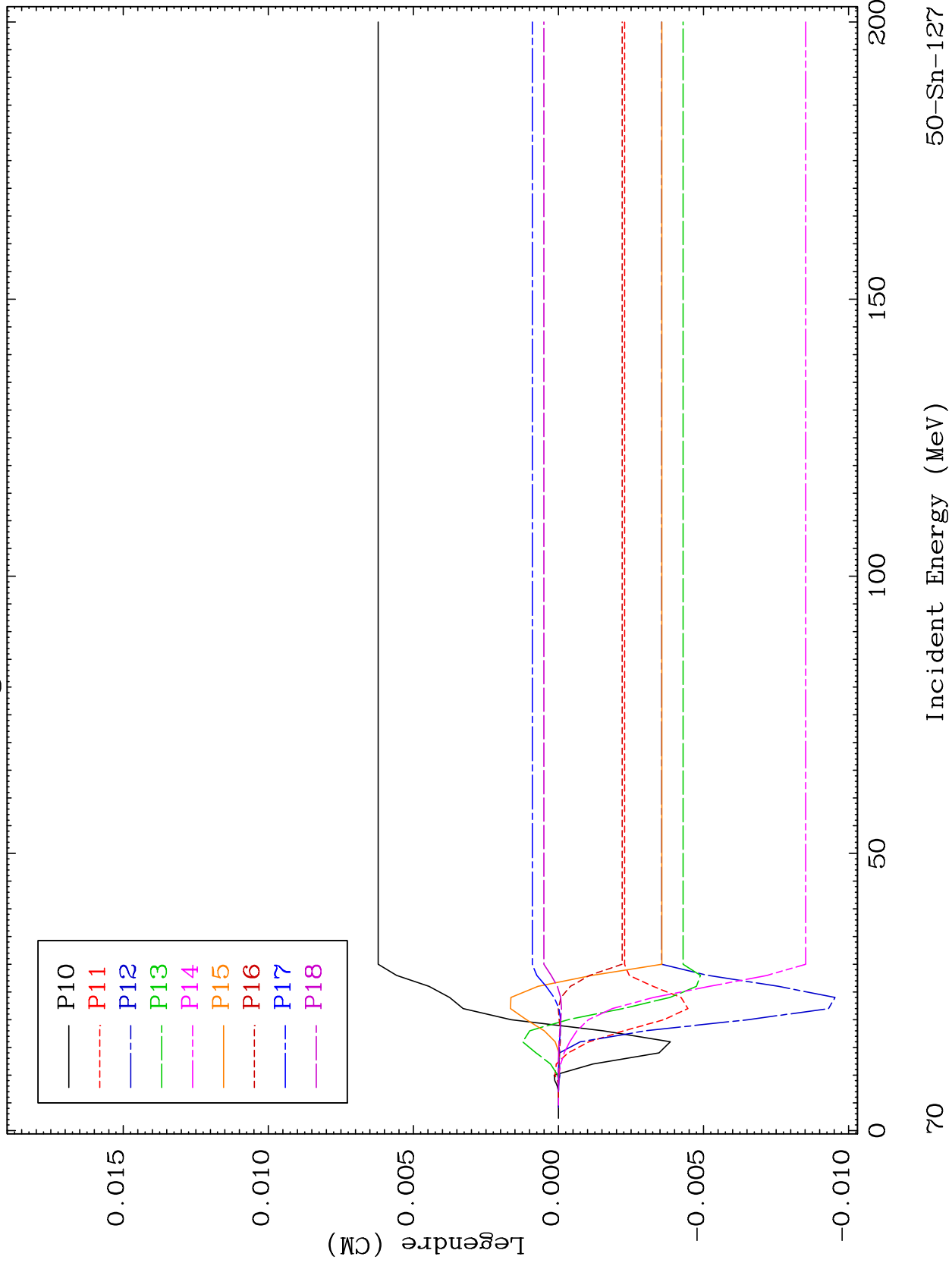


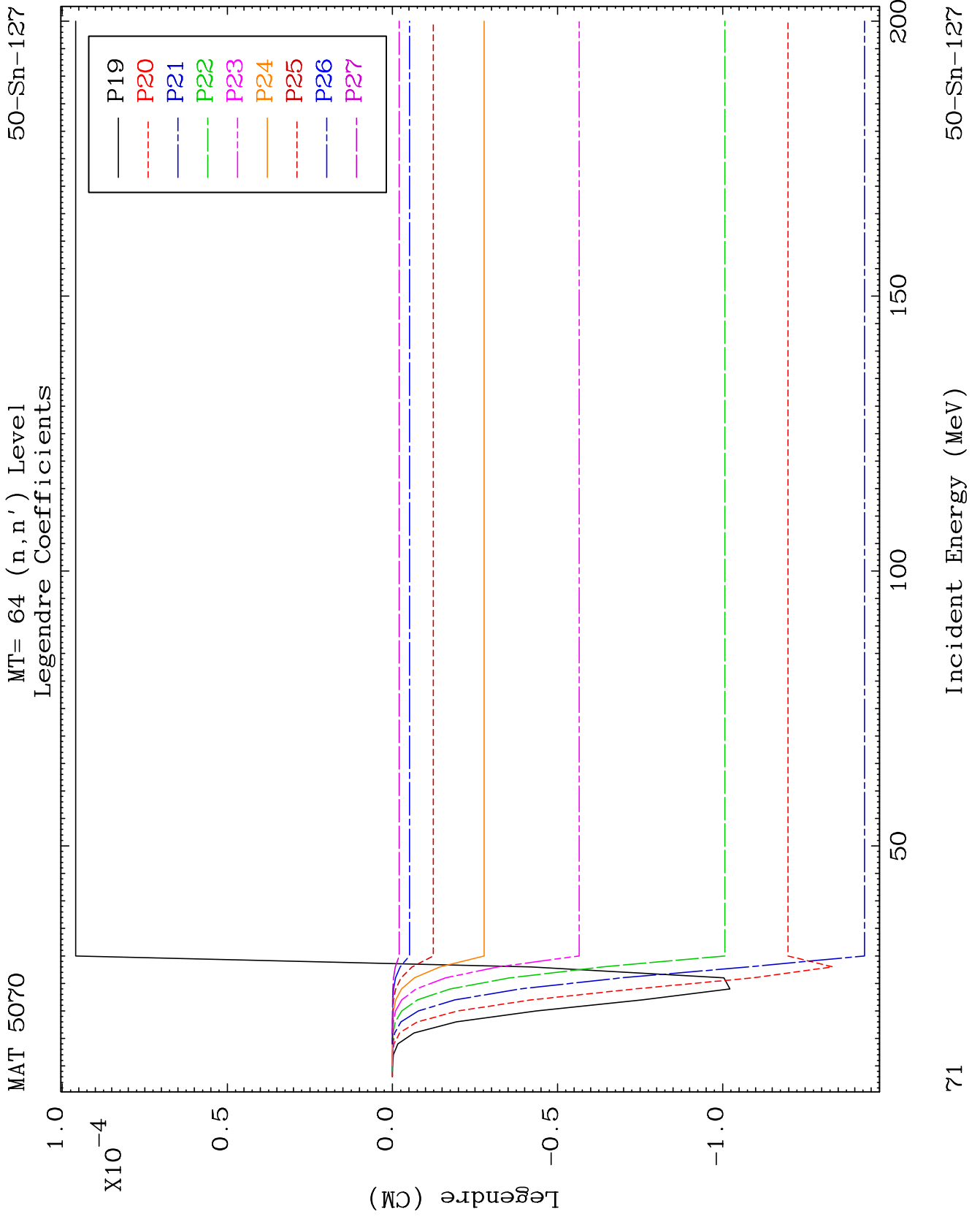


MAT 5070

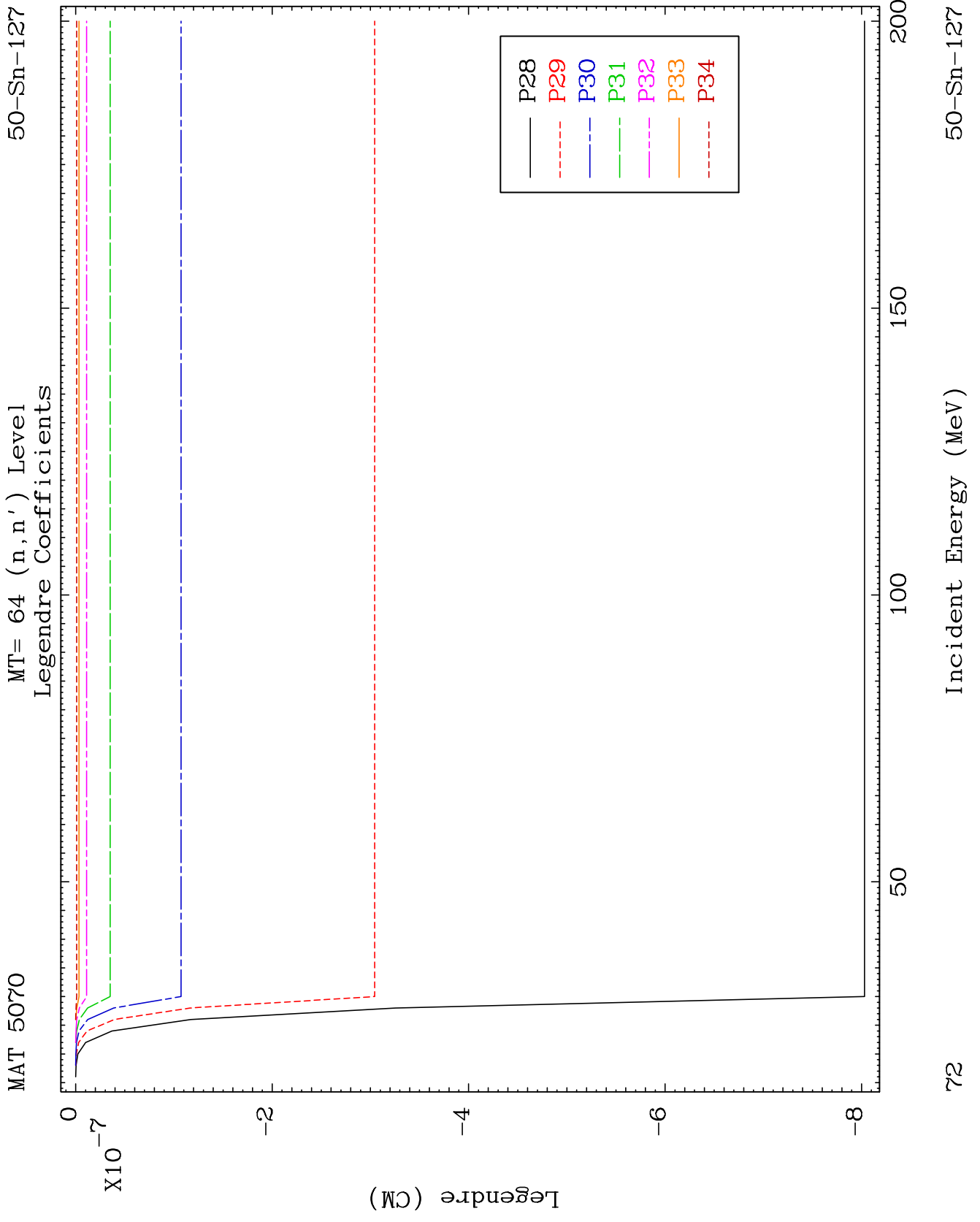
MT= 64 (n,n') Level  
Legendre Coefficients

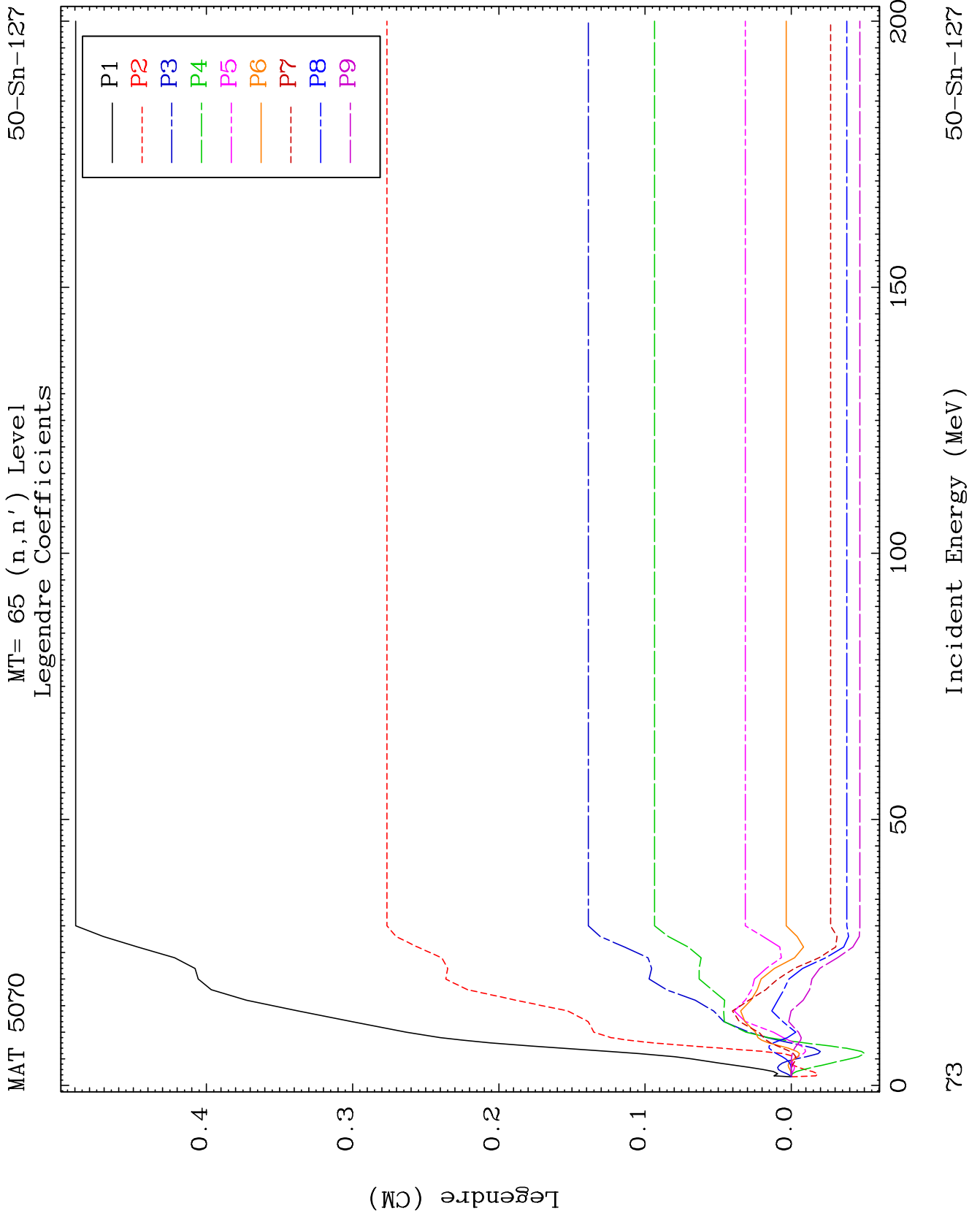
50-Sn-127

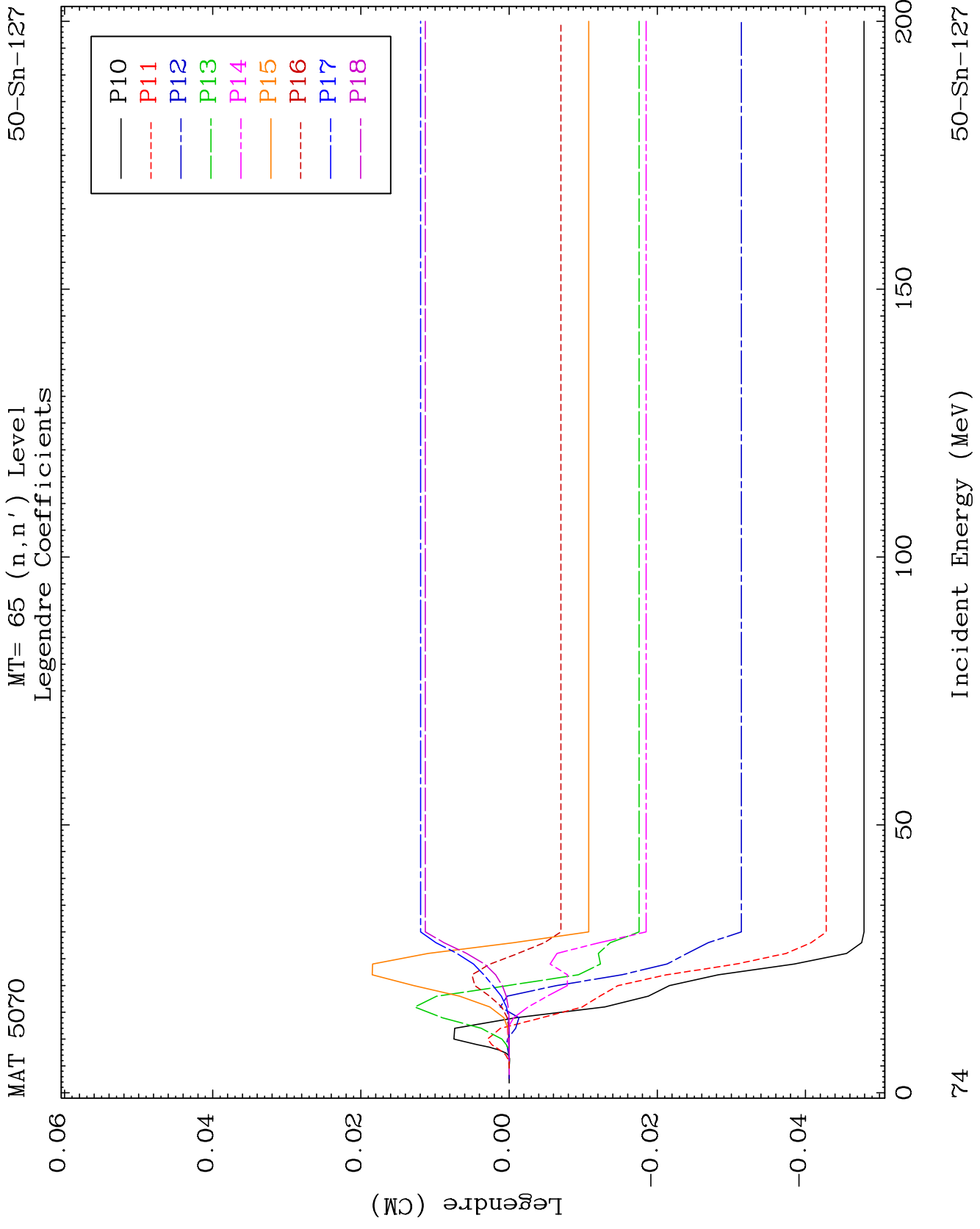


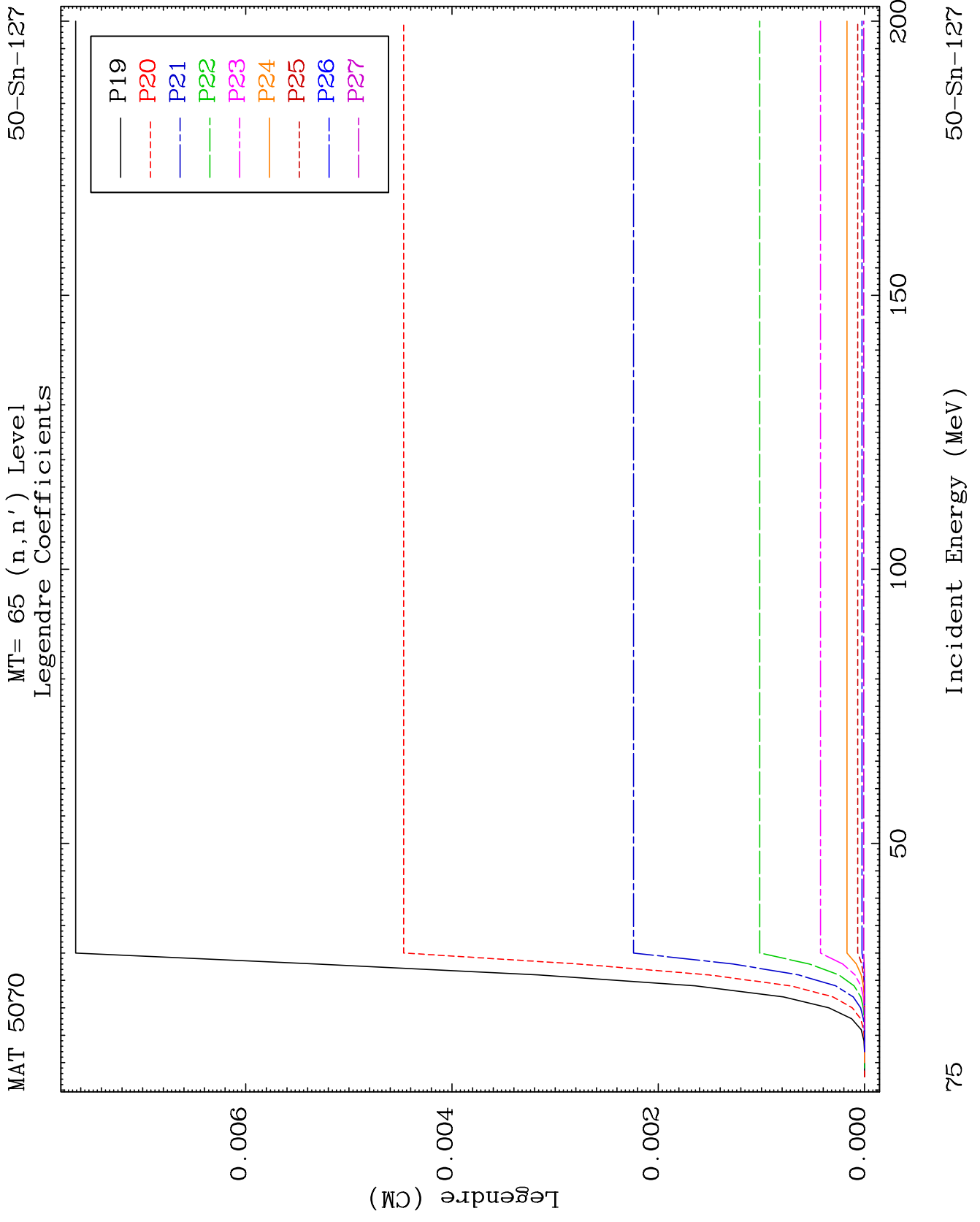


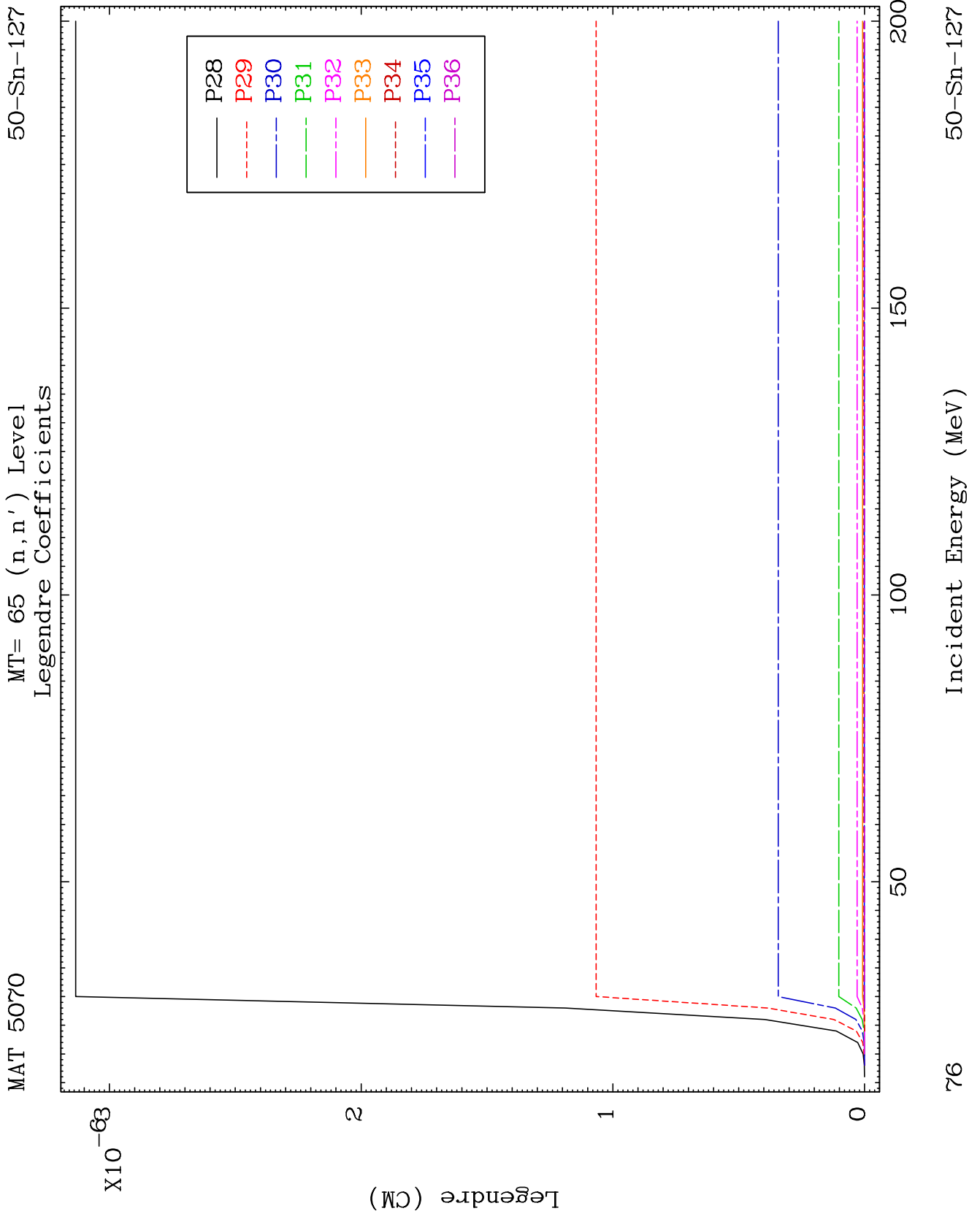








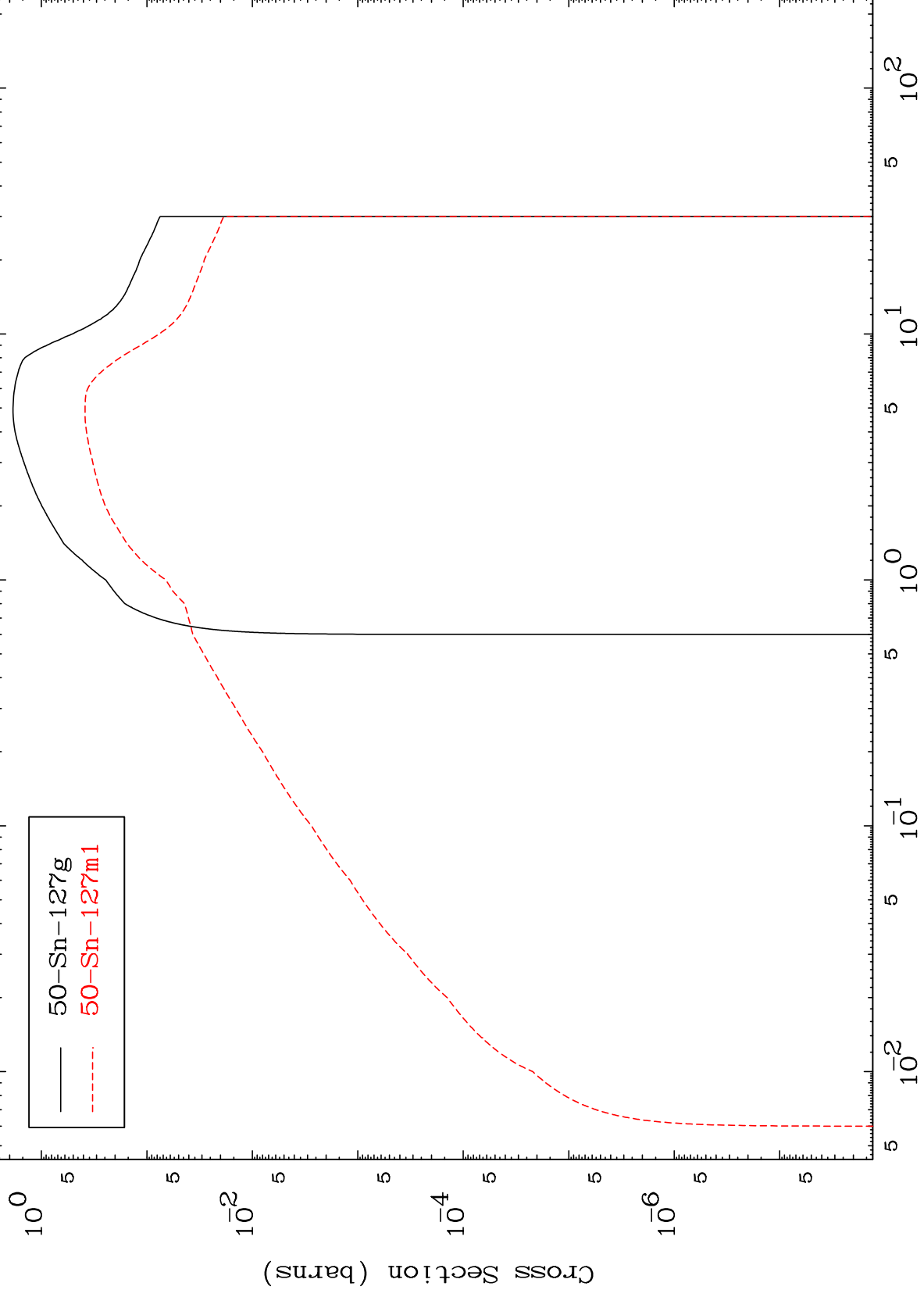




MAT 5070

Radionuclide Production Cross Section

50-Sn-127



50-Sn-127g  
50-Sn-127m1

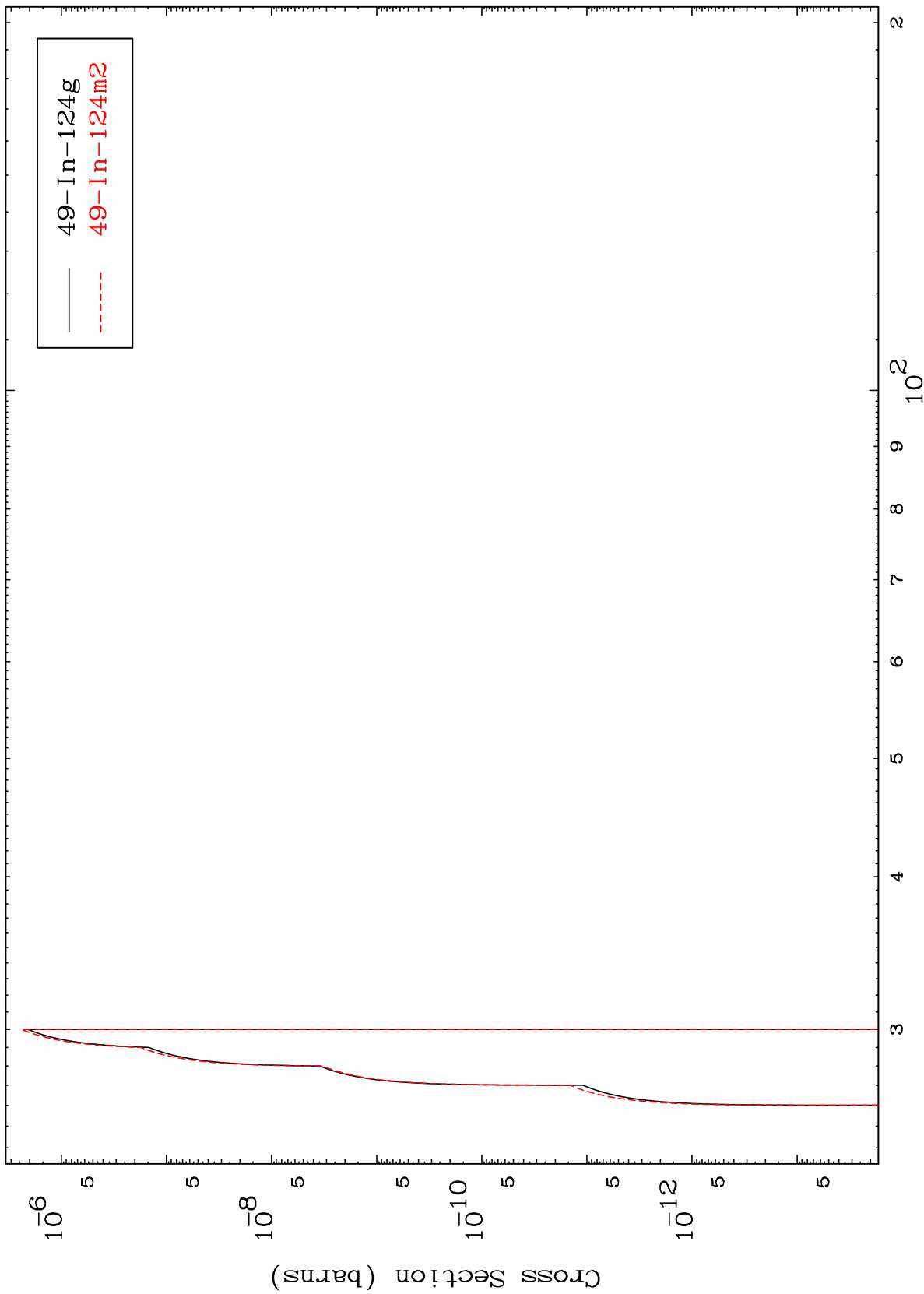
77

50-Sn-127

MAT 5070

50-Sn-127

(n,2n) d  
Radionuclide Production Cross Section



78

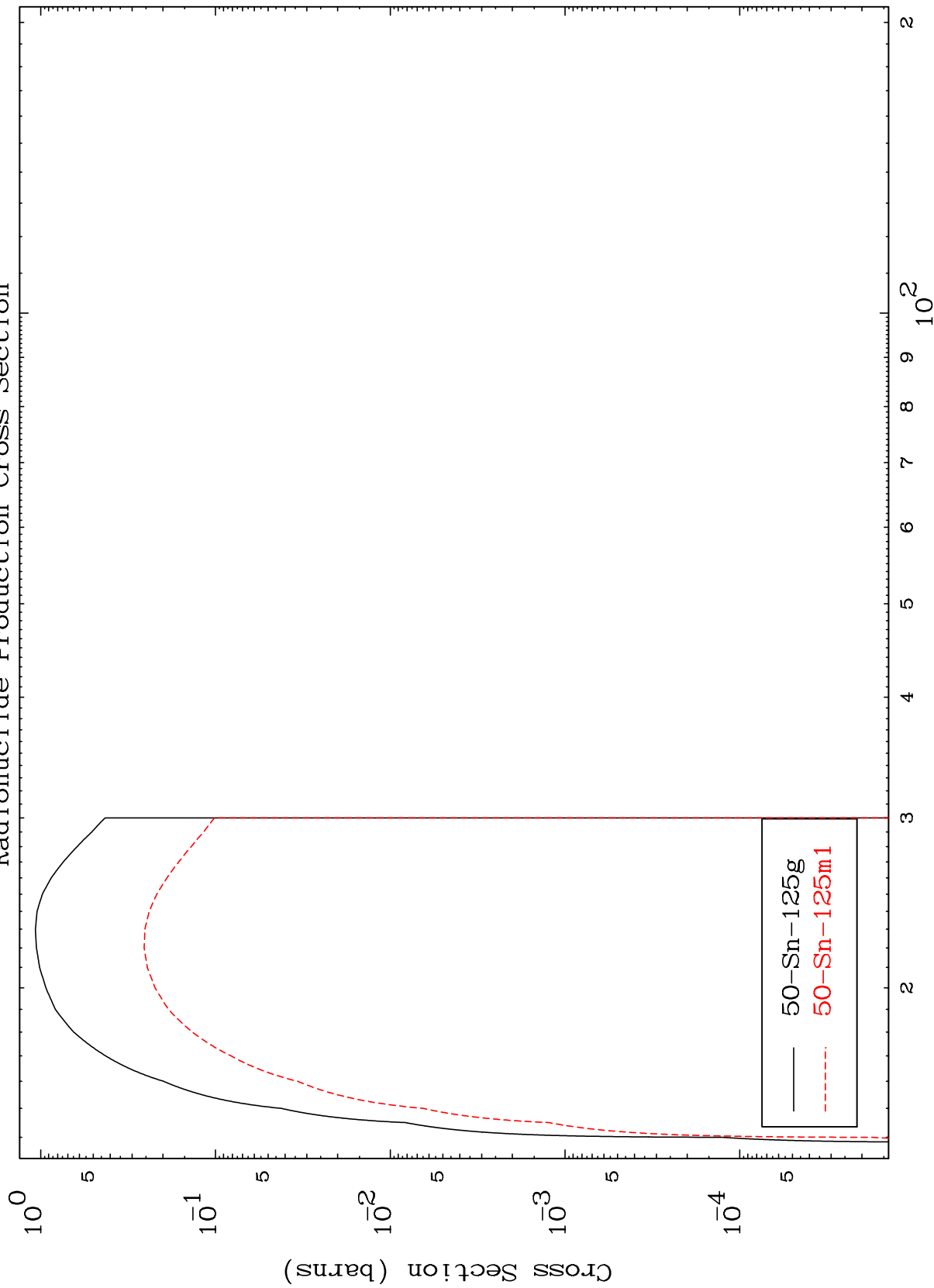
50-Sn-127

Incident Energy (MeV)

MAT 5070

50-Sn-127

Radionuclide Production Cross Section  
(n,3n)



Incident Energy (MeV)

50-Sn-127

79

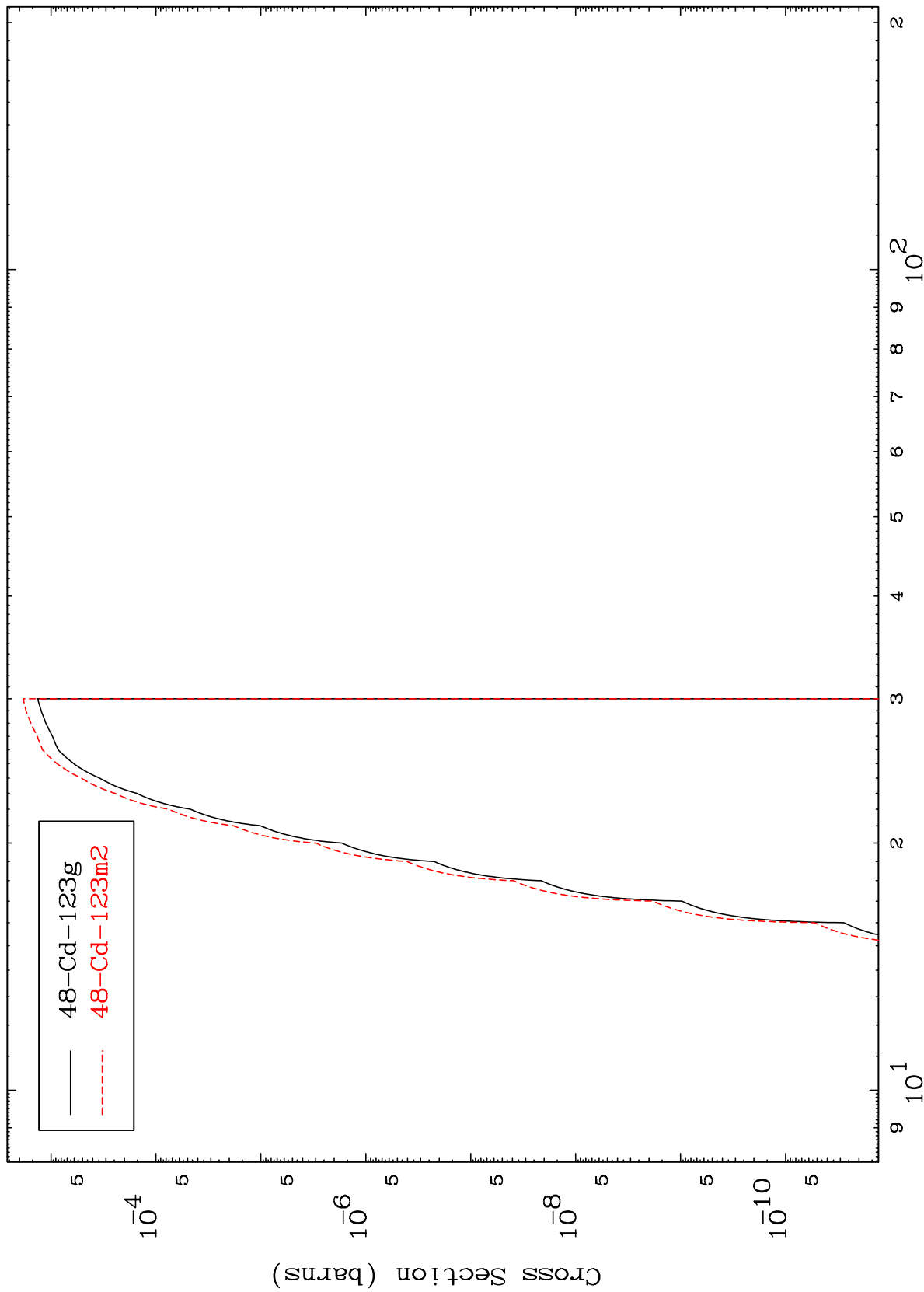


MAT 5070

50-Sn-127

(n,n')  $\alpha$

Radionuclide Production Cross Section

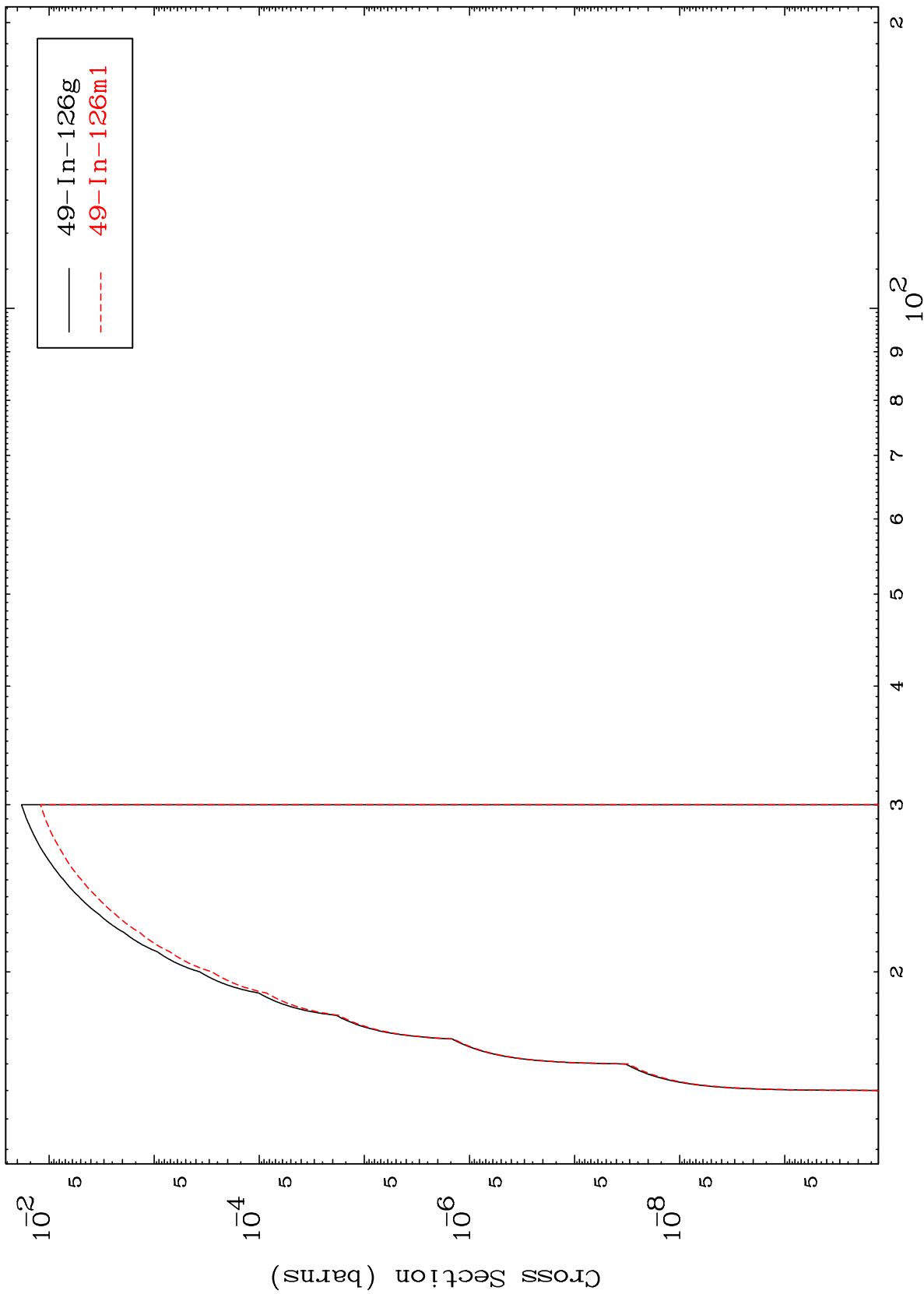


80

Incident Energy (MeV)

50-Sn-127

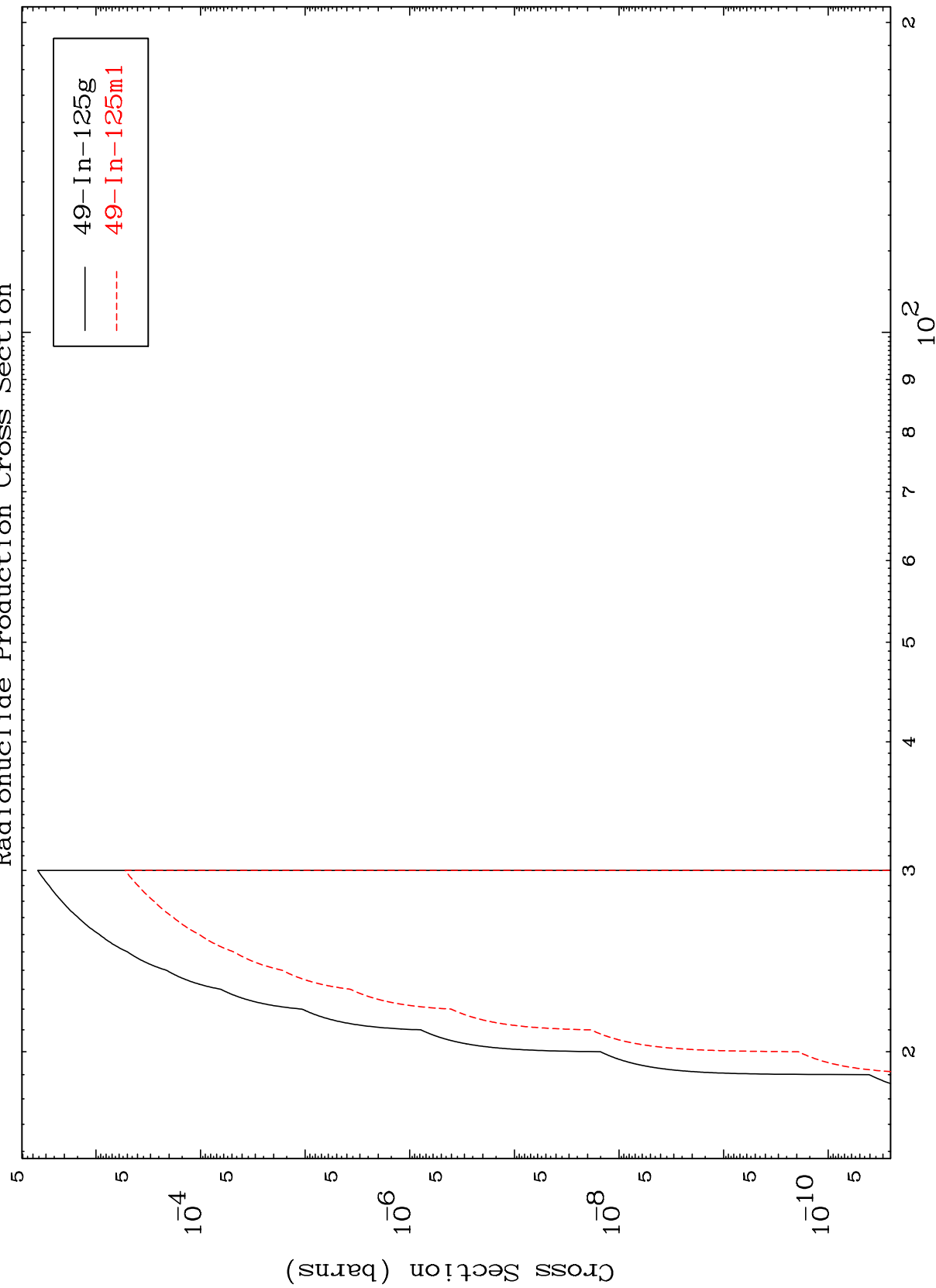
$(n, n')$  p  
Radionuclide Production Cross Section



MAT 5070

50-Sn-127

$(n, n')$  d  
Radionuclide Production Cross Section

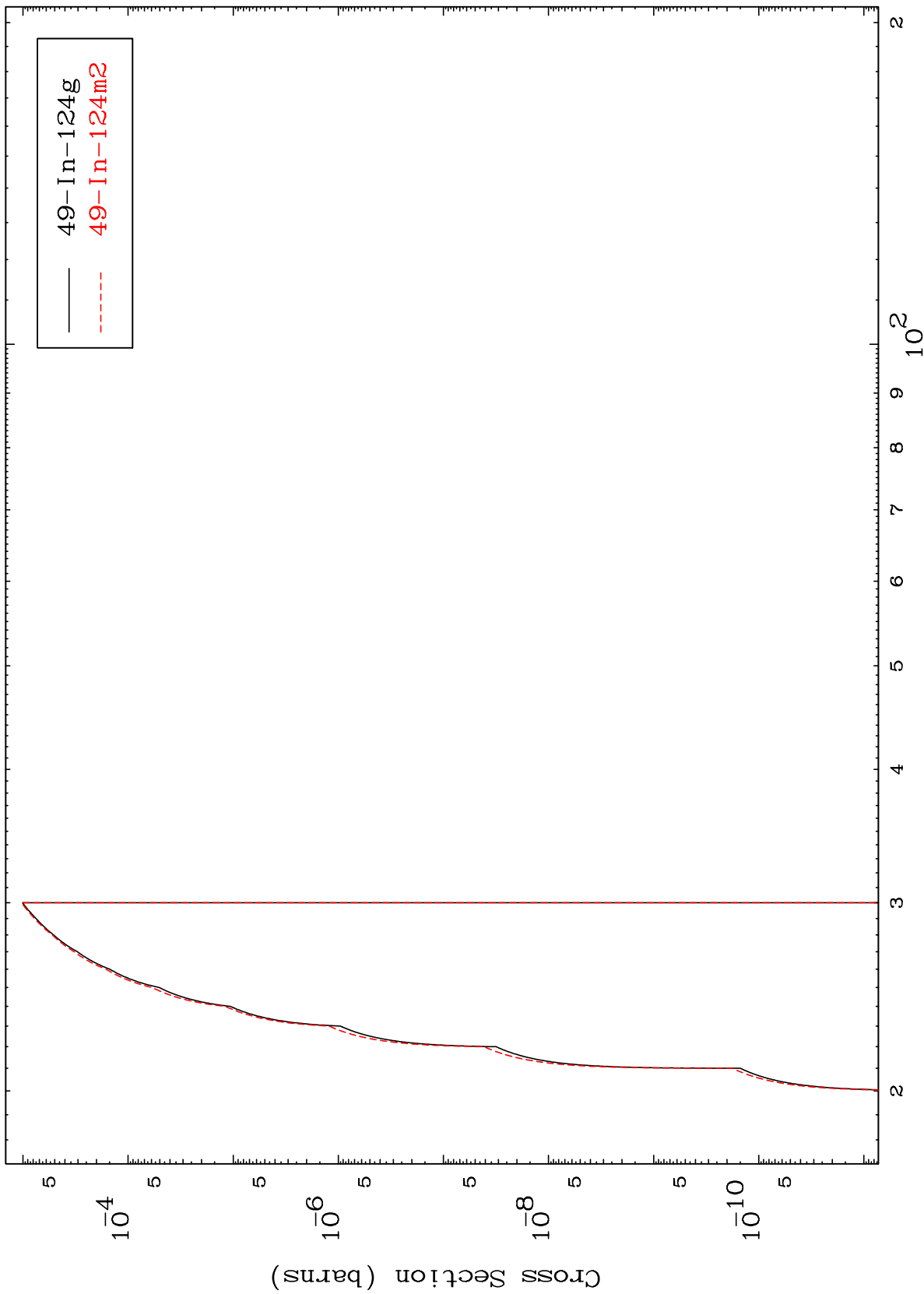


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50-Sn-127

Incident Energy (MeV)

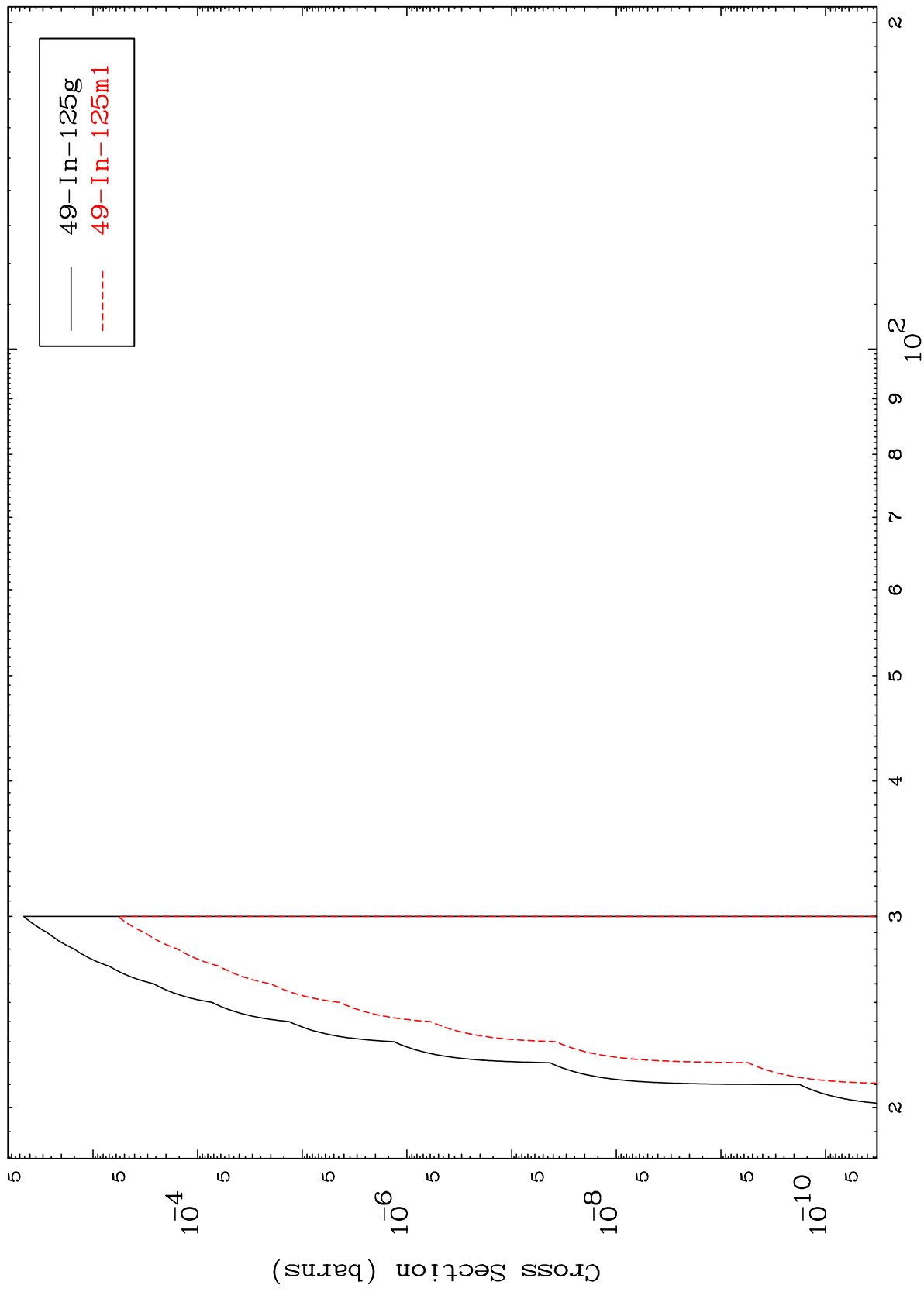
Radionuclide Production Cross Section



MAT 5070

50-Sn-127

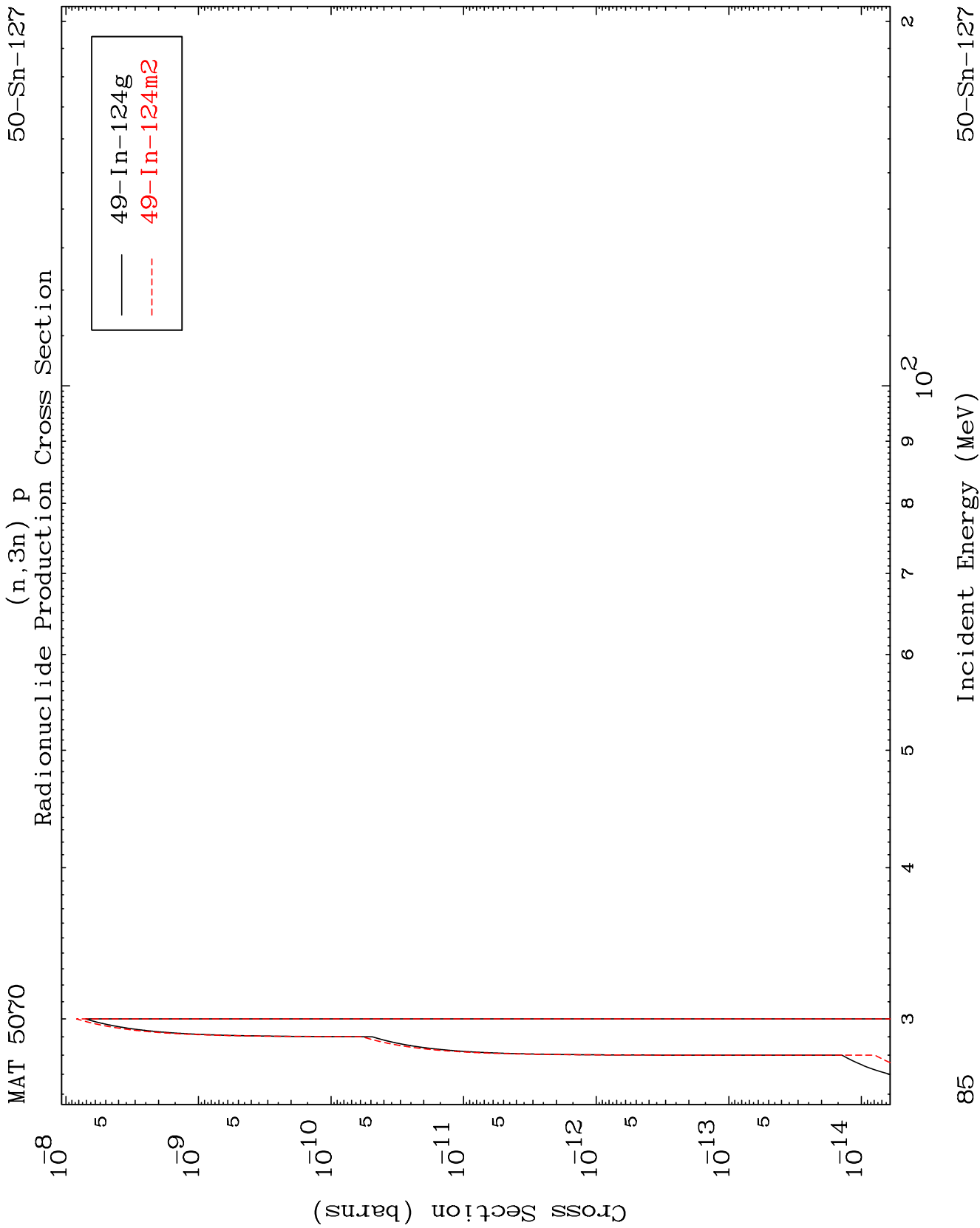
(n,2n) p  
Radionuclide Production Cross Section



84

Incident Energy (MeV)

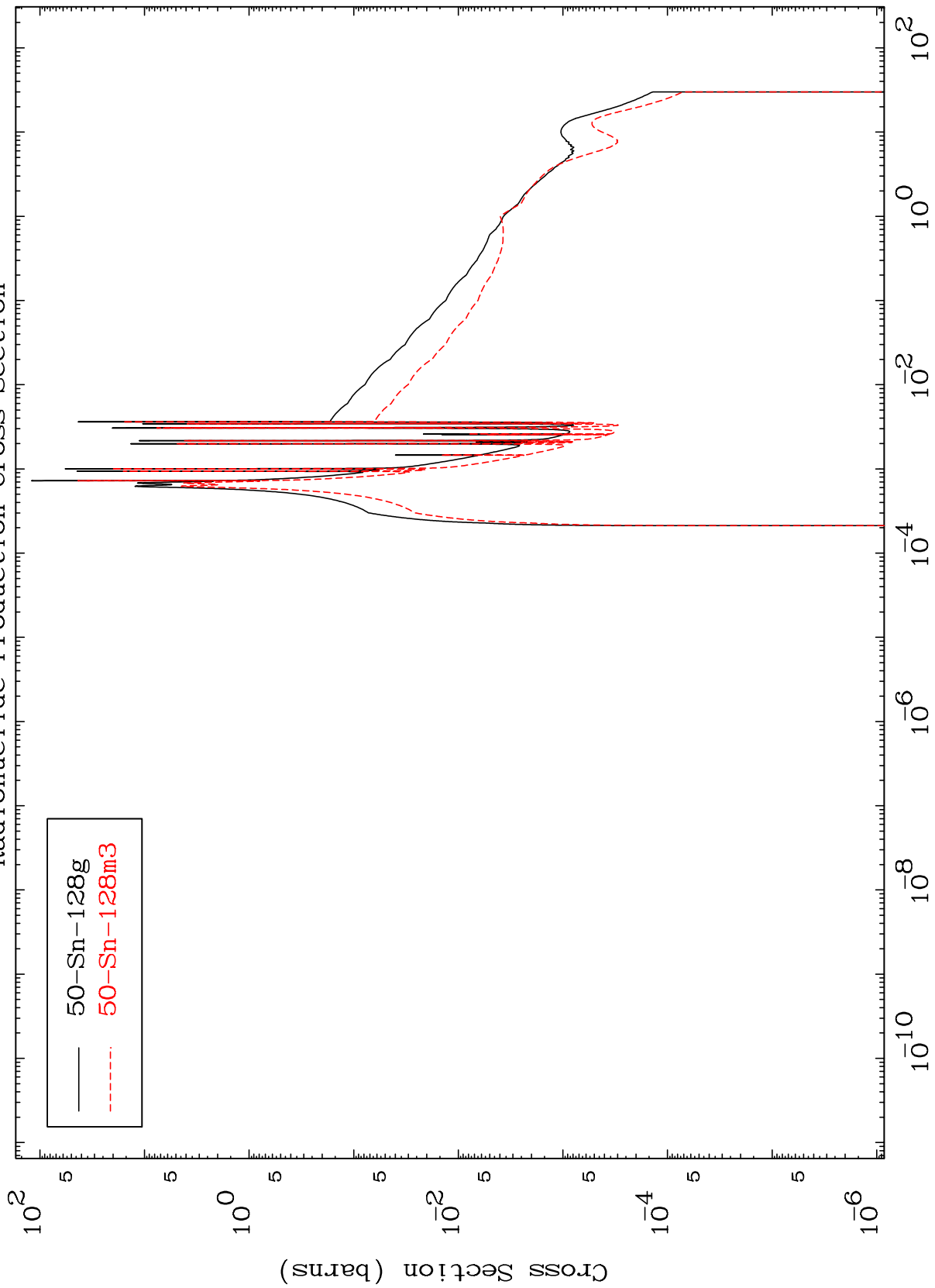
50-Sn-127



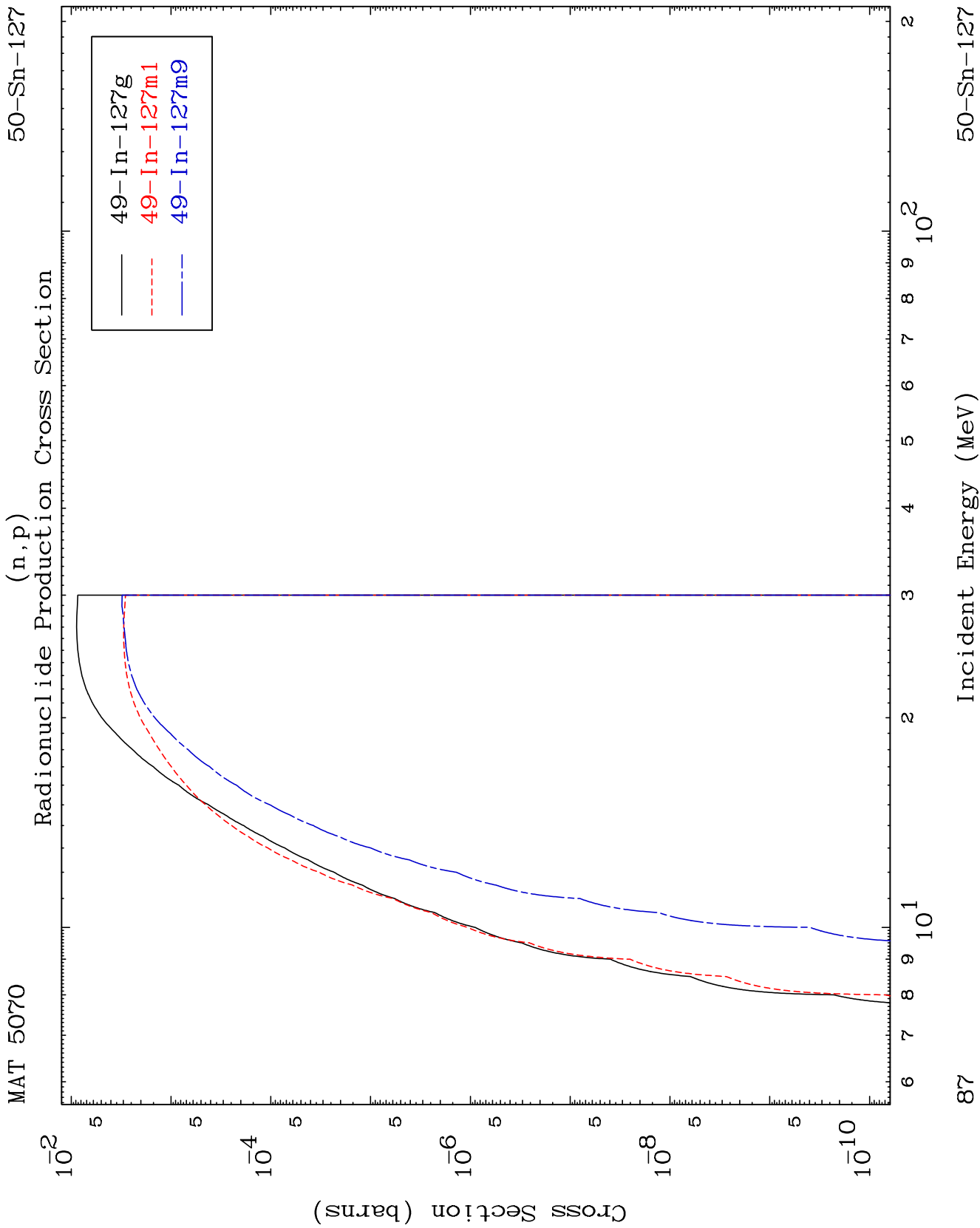
MAT 5070

50-Sn-127

Radionuclide Production Cross Section



50-Sn-127



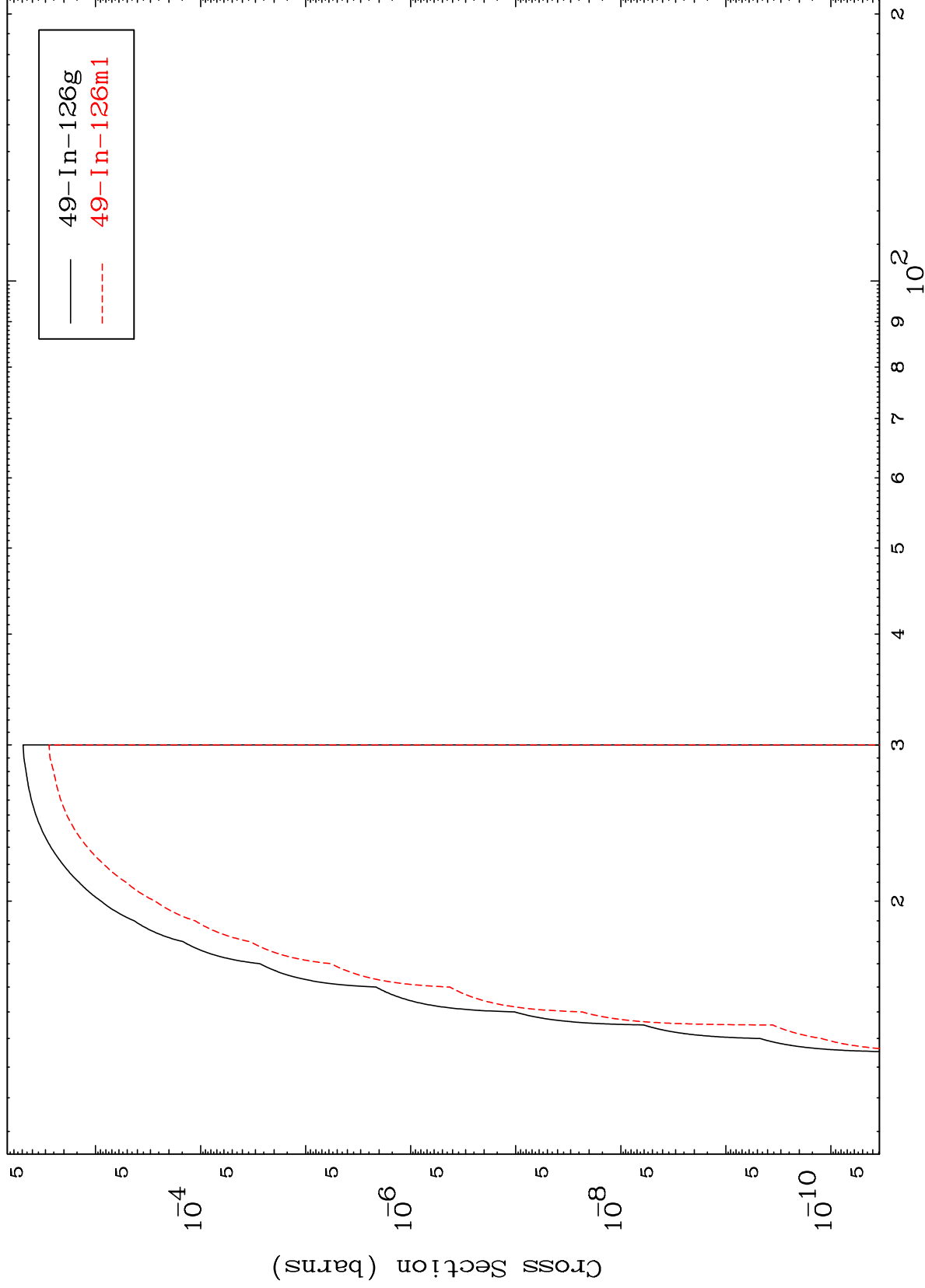


MAT 5070

(n,d)

50-Sn-127

Radionuclide Production Cross Section



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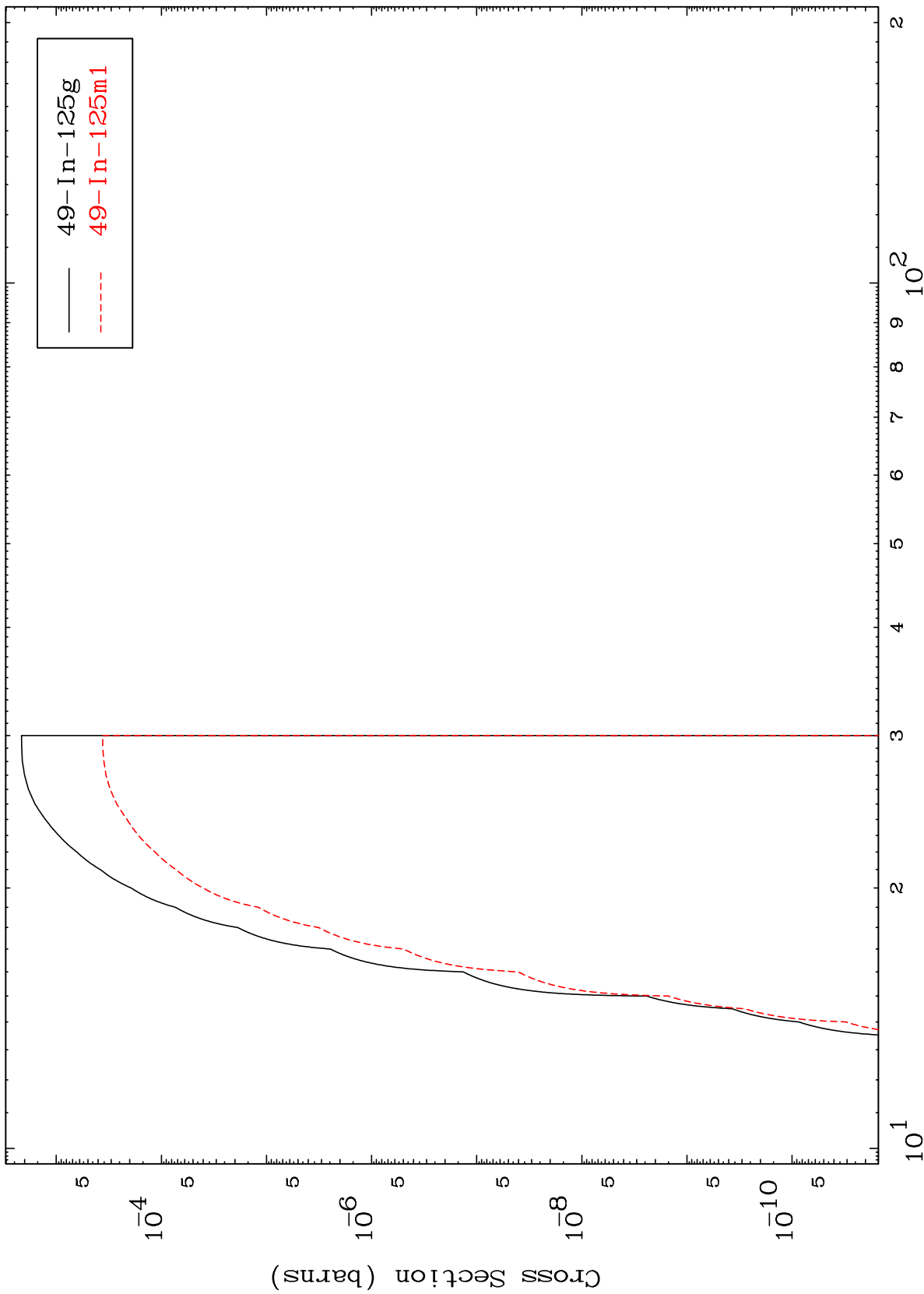
Incident Energy (MeV)

50-Sn-127

MAT 5070

50-Sn-127

(n,t)  
Radionuclide Production Cross Section



89

Incident Energy (MeV)

50-Sn-127