

Program EVALPLOT  
(Version 2021-1)

by

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(Present Contact Information)

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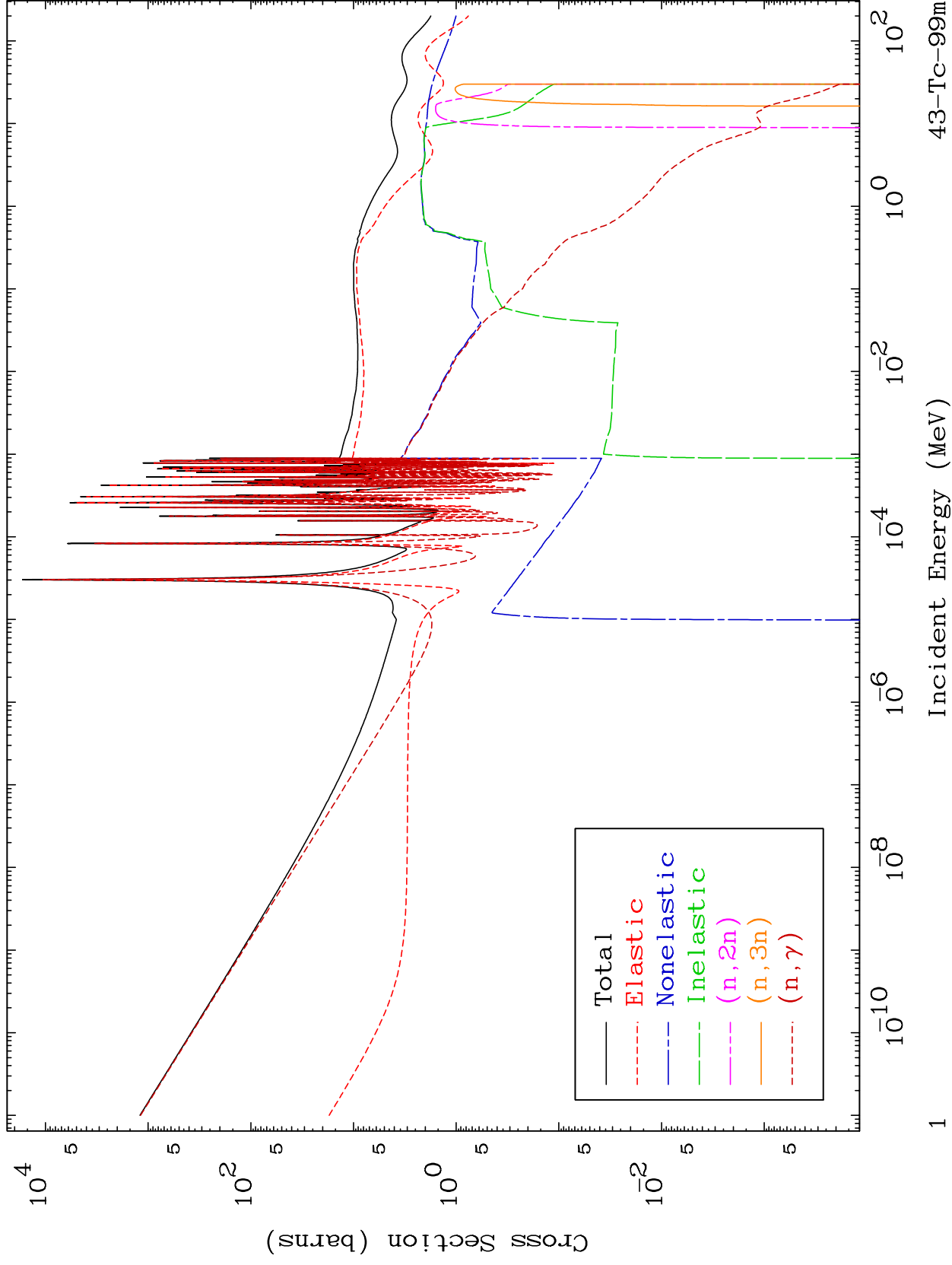
Press Mouse Button to Start

MAT 4326

Neutron Major

43-Tc-99m

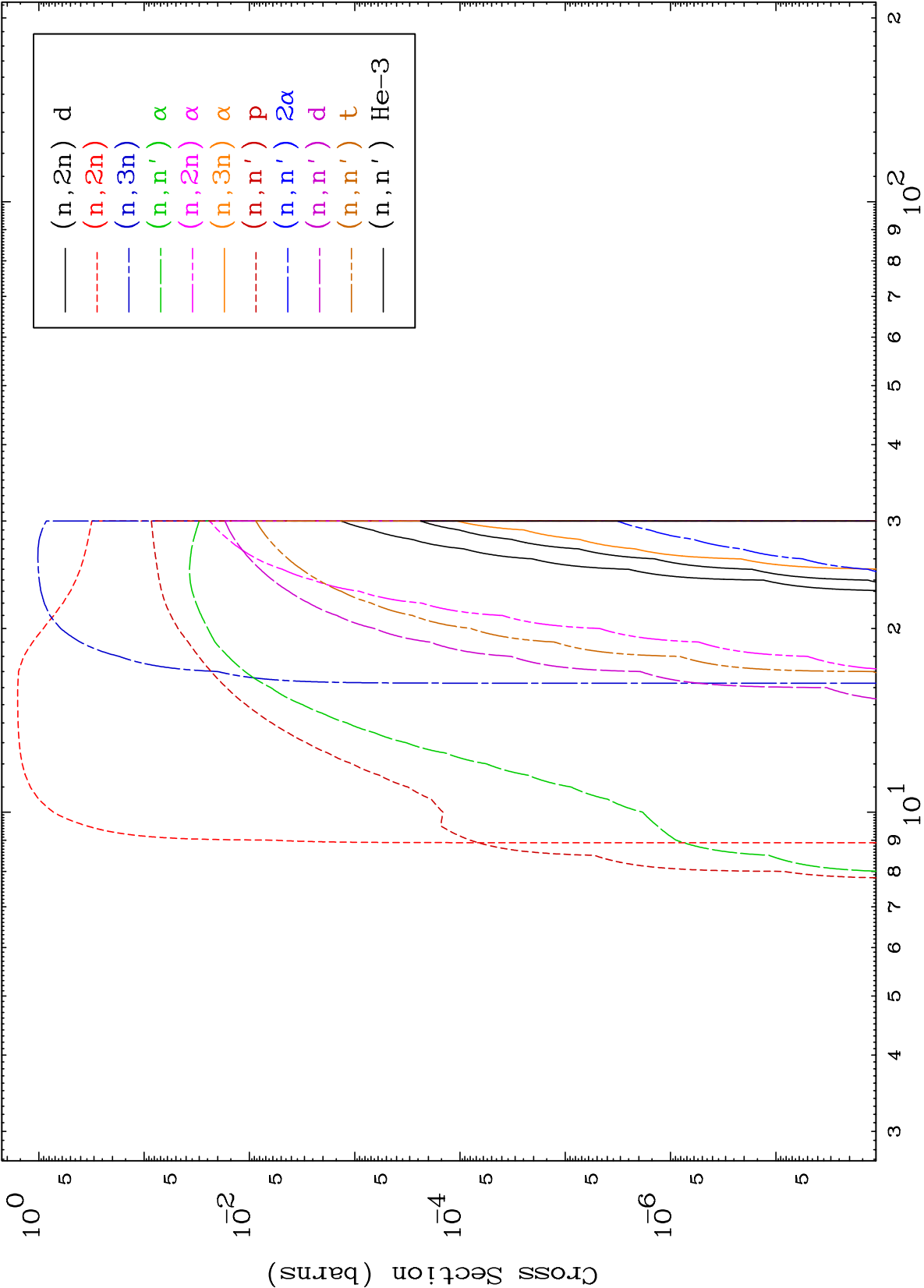
293 Kelvin Cross Sections



MAT 4326

Neutron Absorption  
293 Kelvin Cross Sections

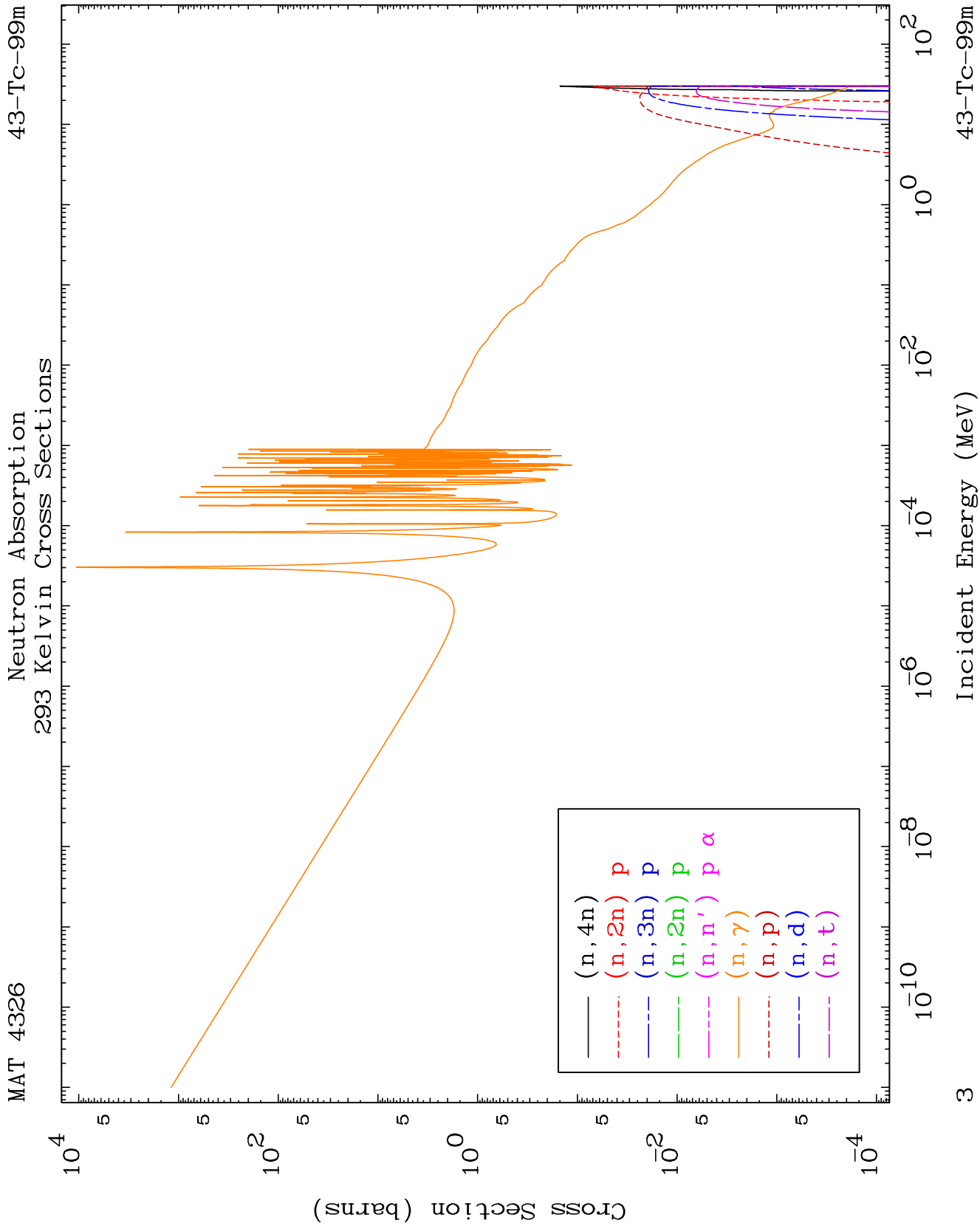
43-Tc-99m

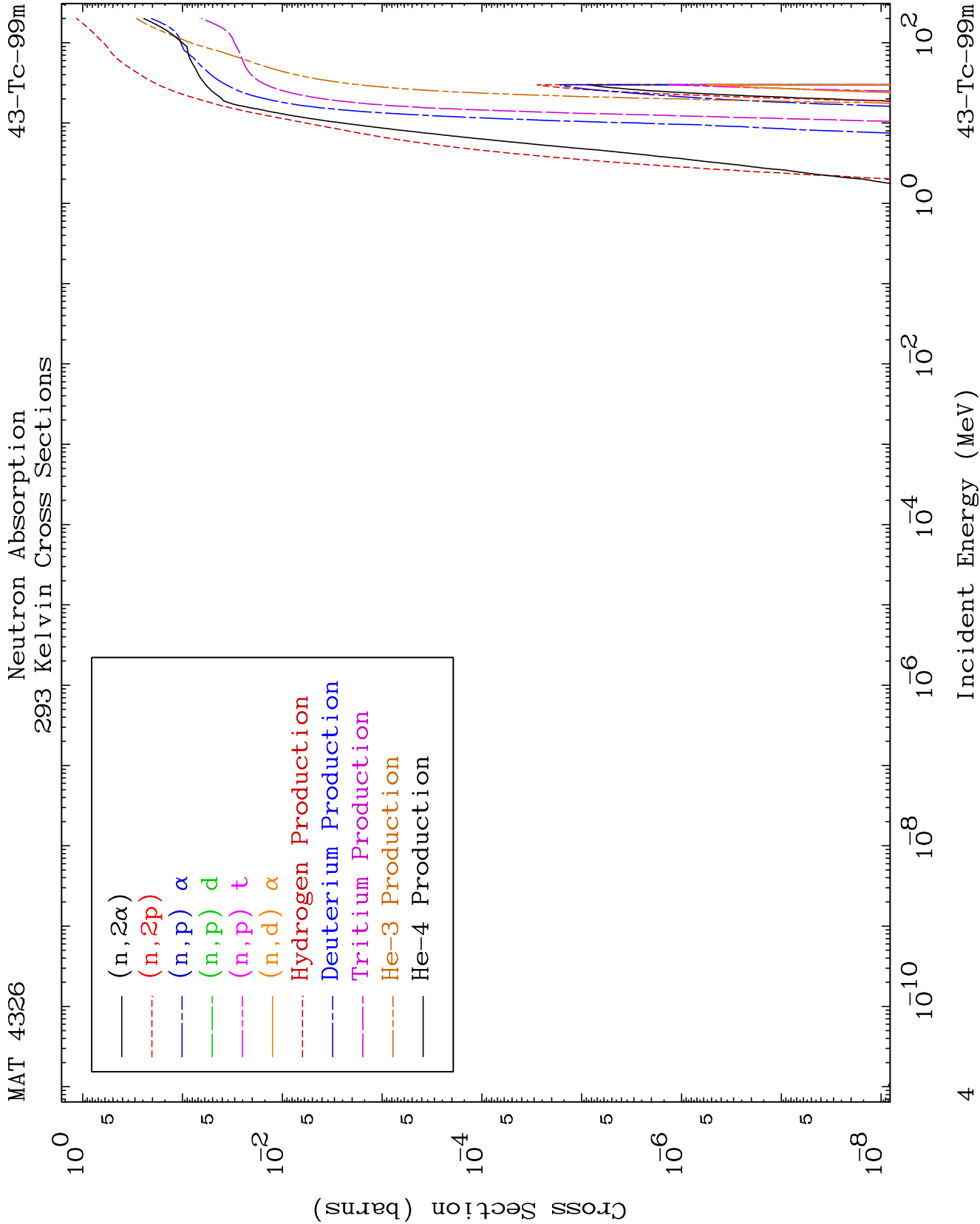


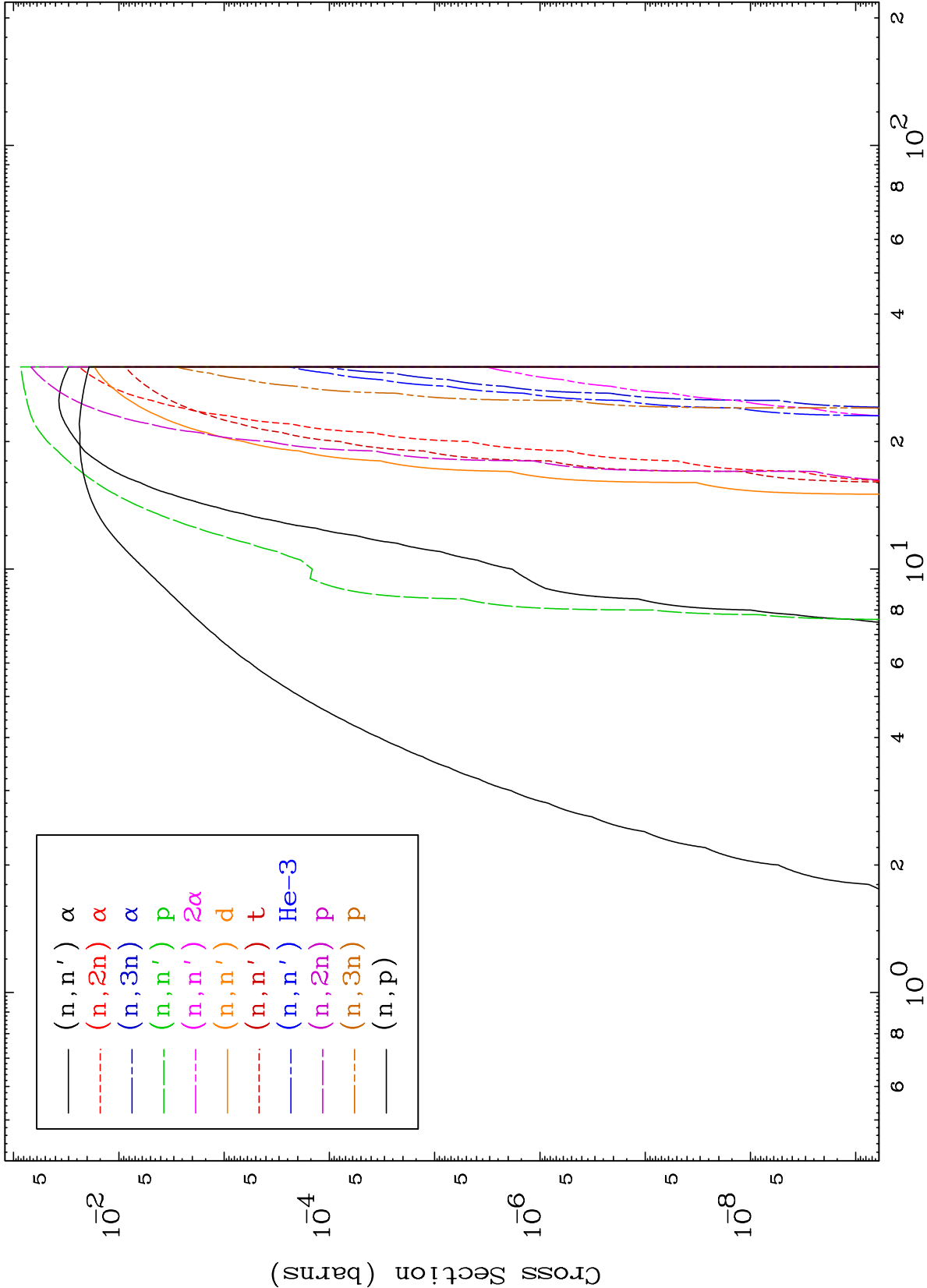
2

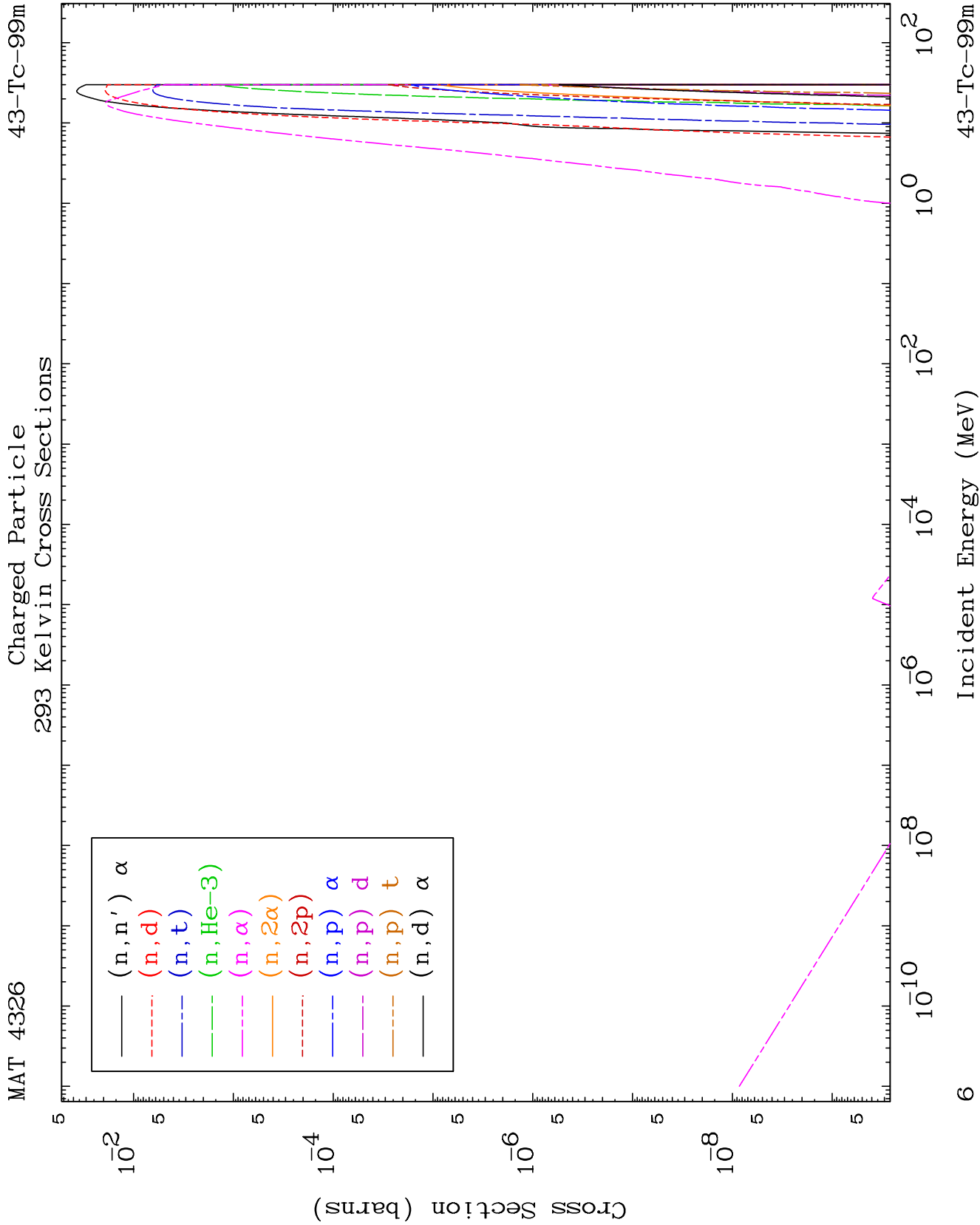
Incident Energy (MeV)

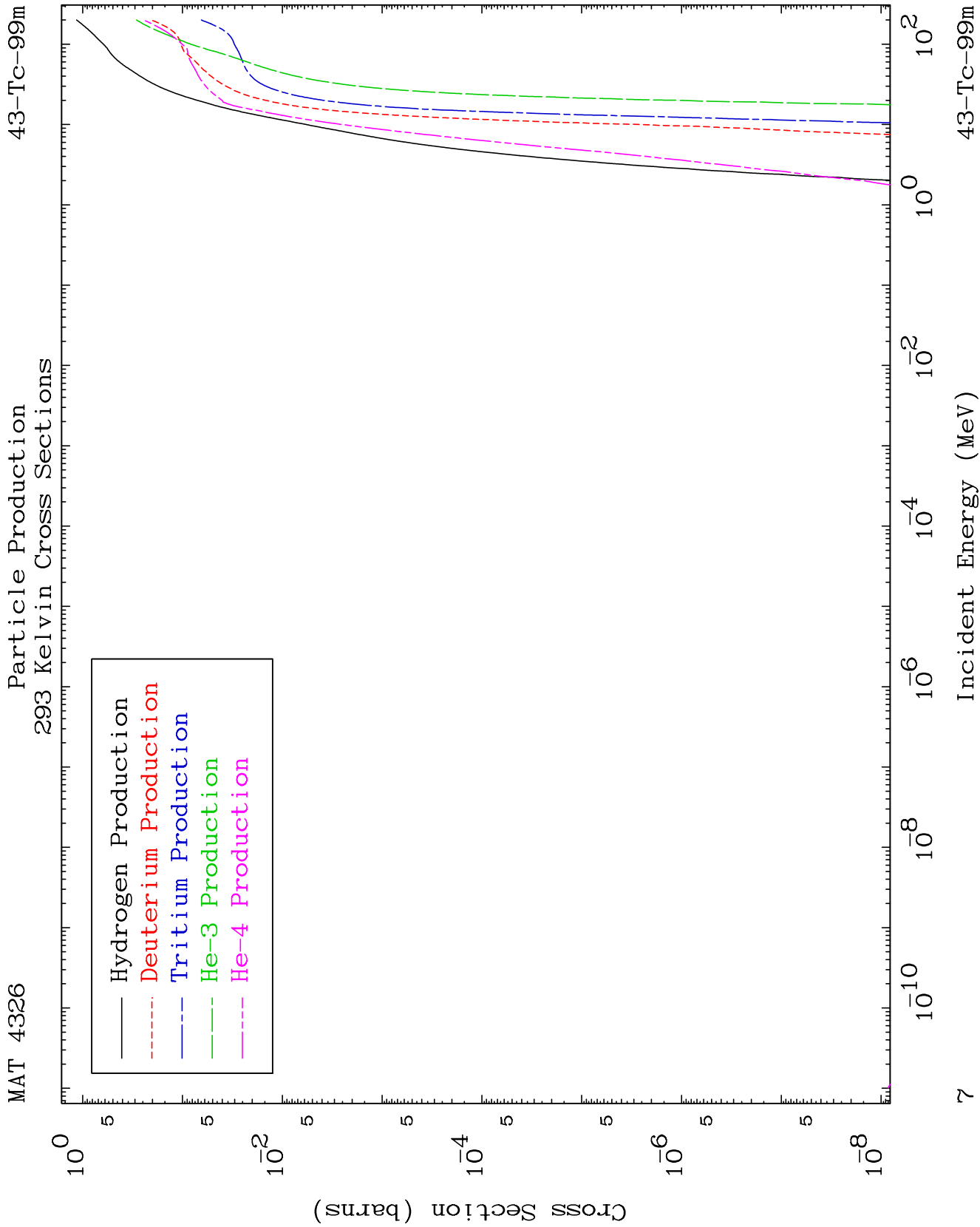
43-Tc-99m







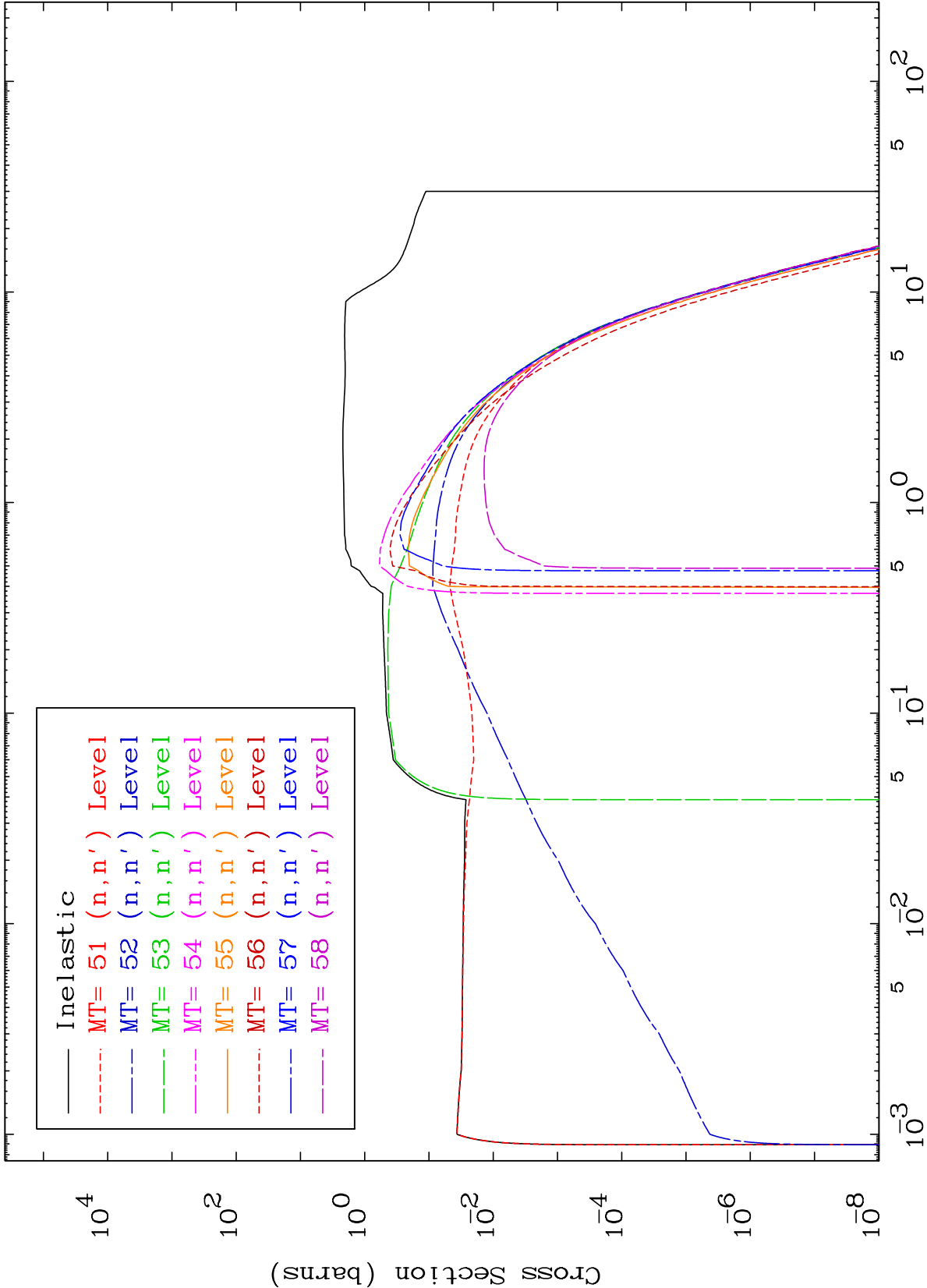




MAT 4326

(n,n') Levels  
293 Kelvin Cross Sections

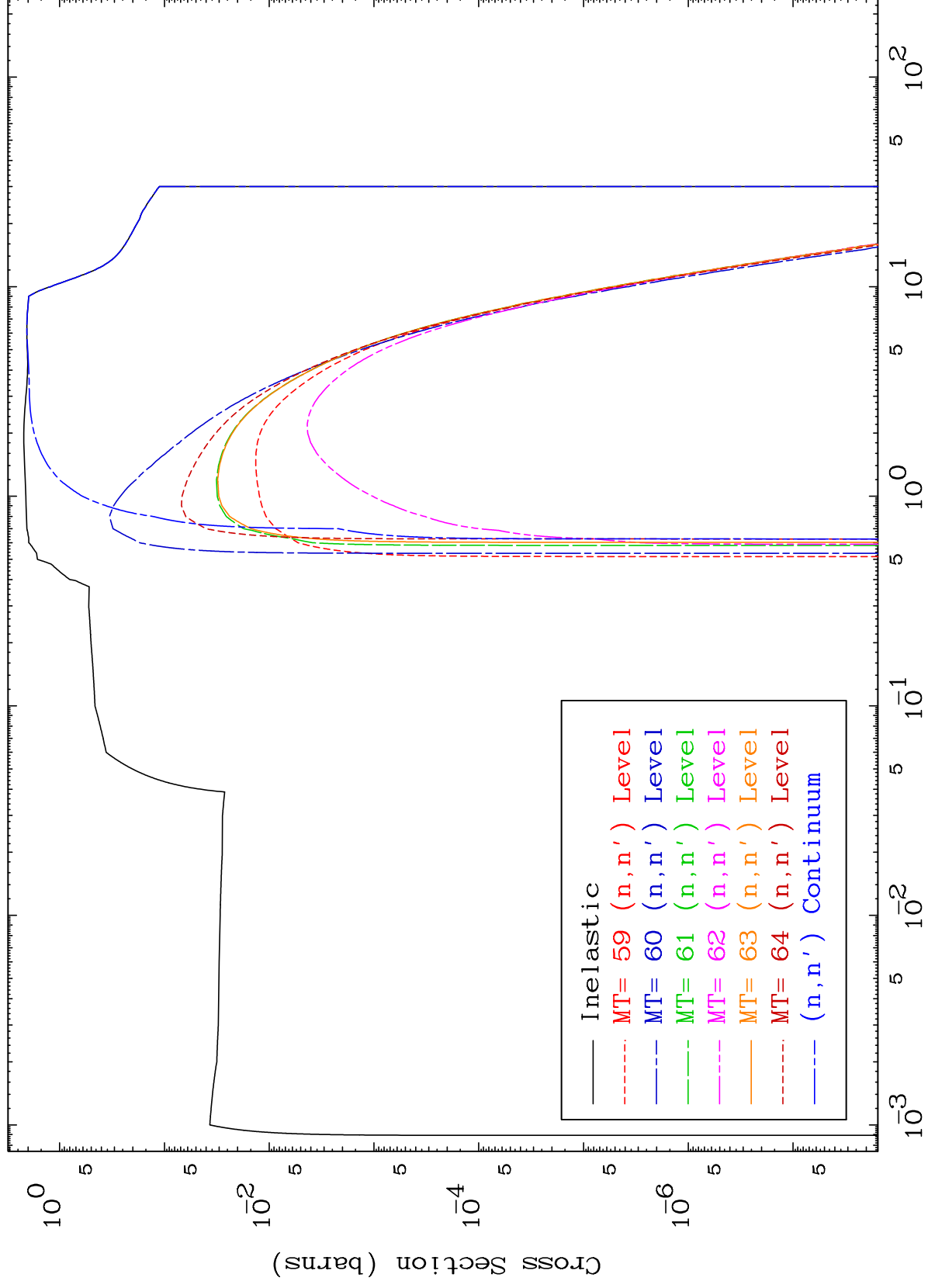
43-Tc-99m



MAT 4326

(n,n') Levels  
293 Kelvin Cross Sections

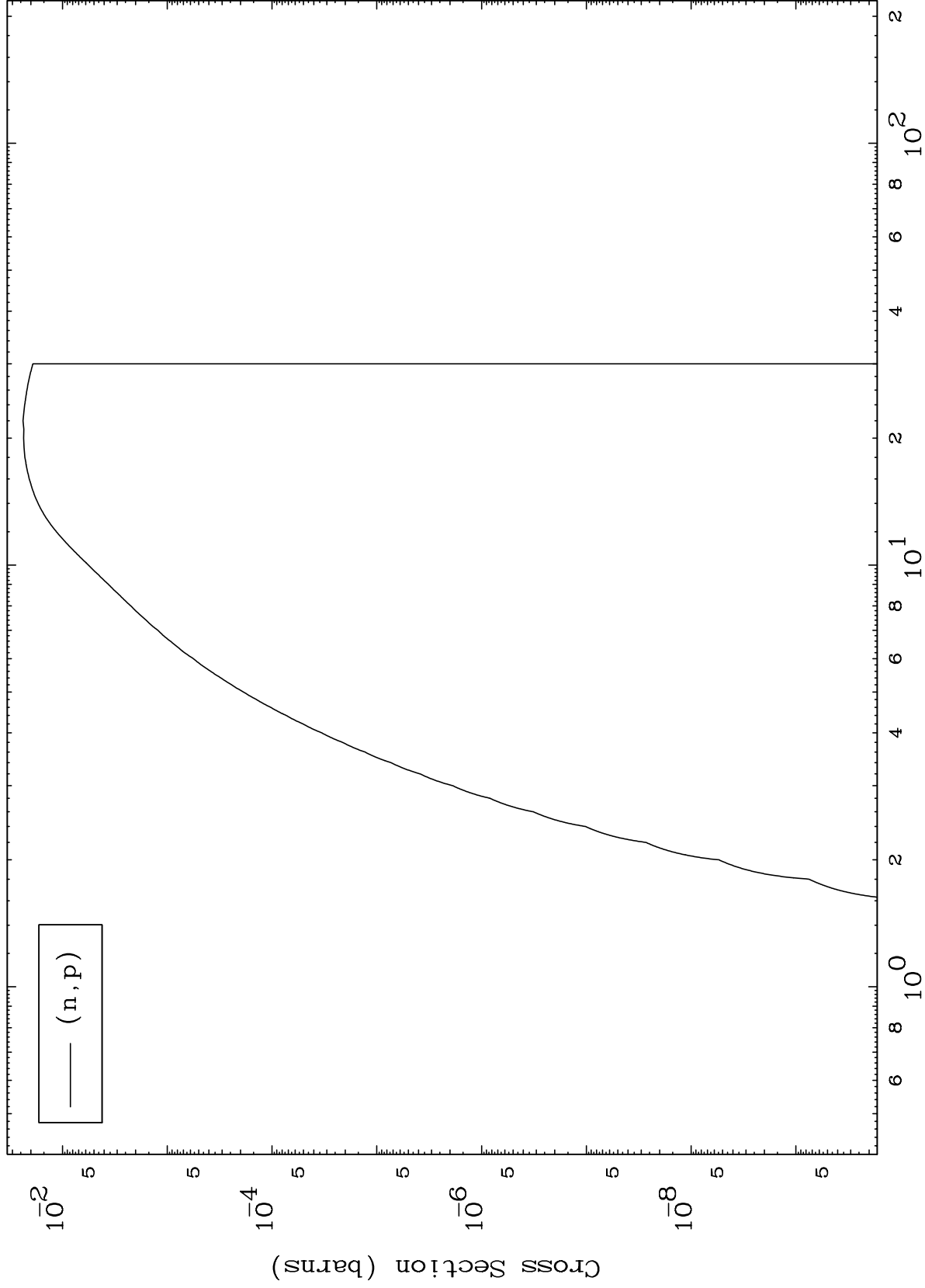
43-Tc-99m



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43-Tc-99m

(n,p) Levels  
293 Kelvin Cross Sections



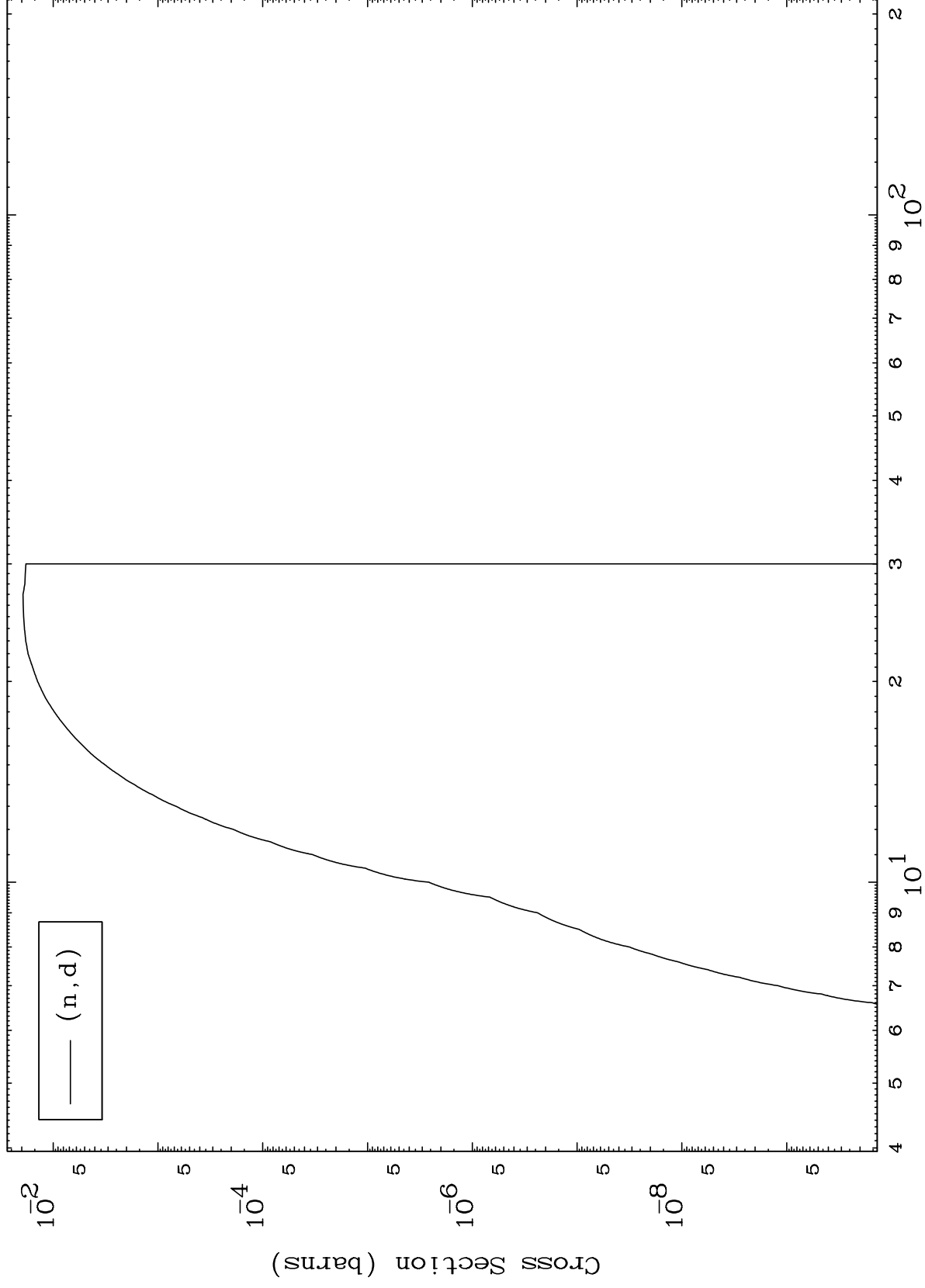
Incident Energy (MeV)

43-Tc-99m

MAT 4326

(n,d) Levels  
293 Kelvin Cross Sections

43-Tc-99m



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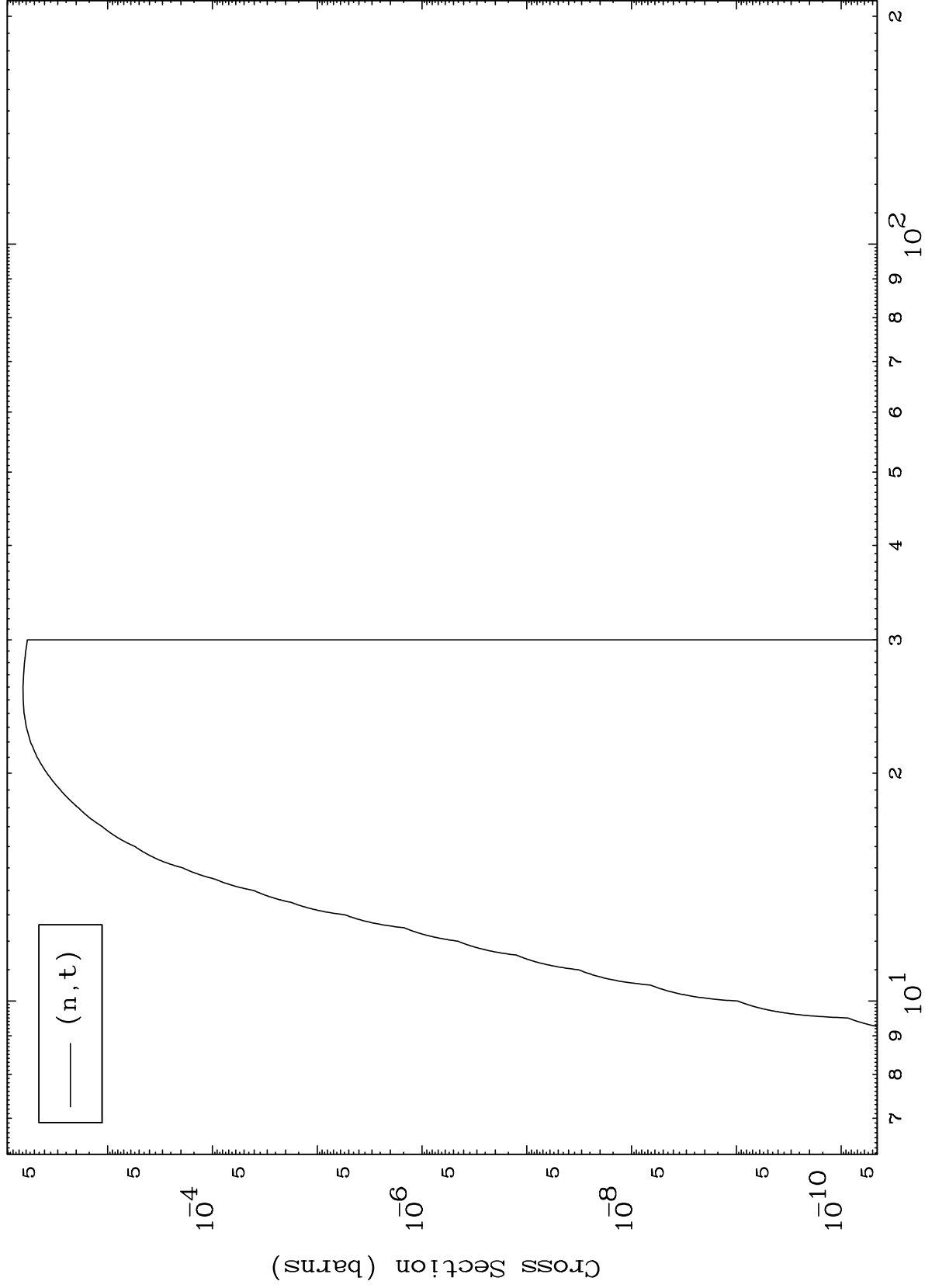
Incident Energy (MeV)

43-Tc-99m

MAT 4326

(n,t) Levels  
293 Kelvin Cross Sections

43-Tc-99m



12

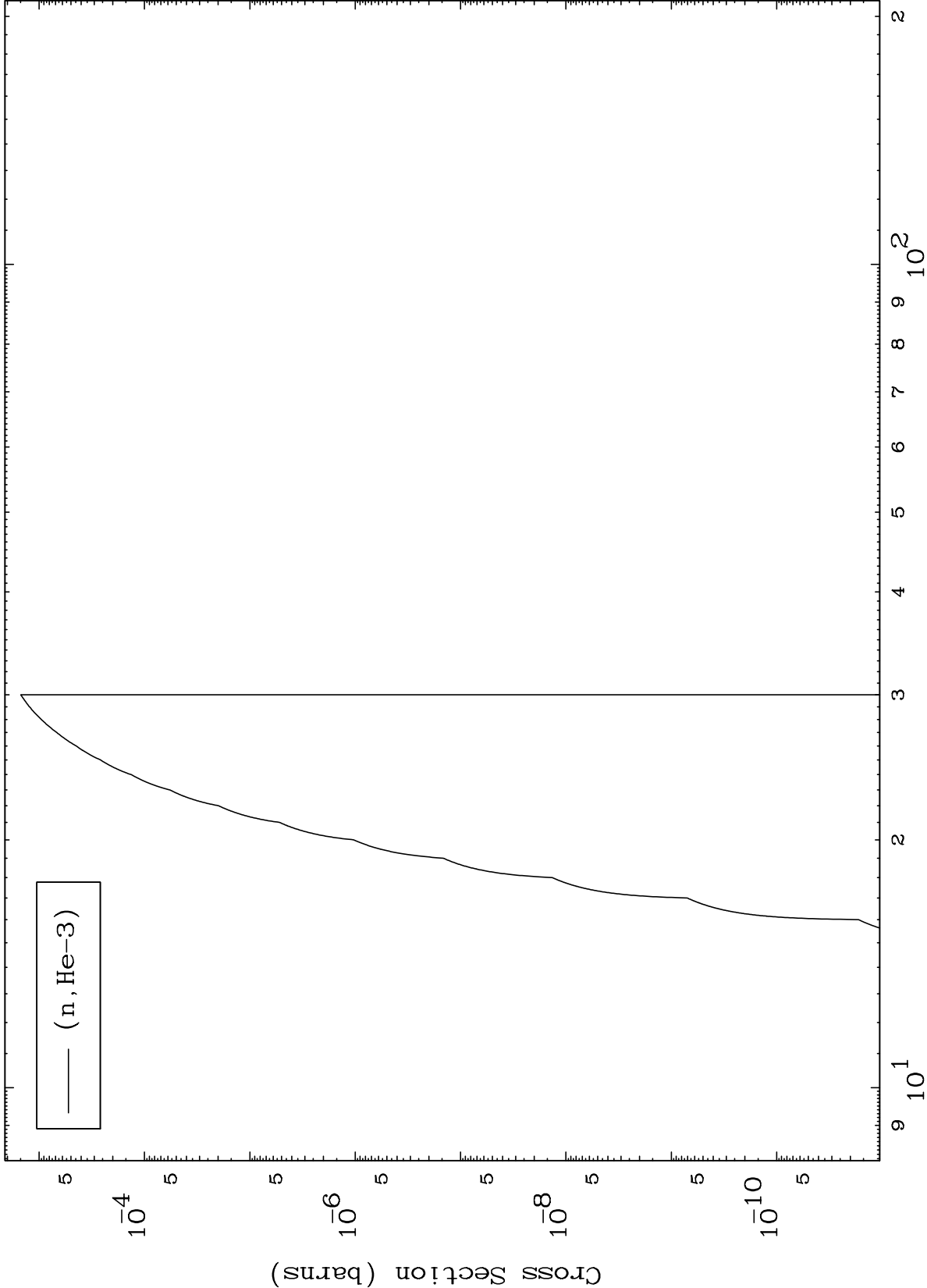
Incident Energy (MeV)

43-Tc-99m

MAT 4326

(n,He3) Levels  
293 Kelvin Cross Sections

43-Tc-99m



13

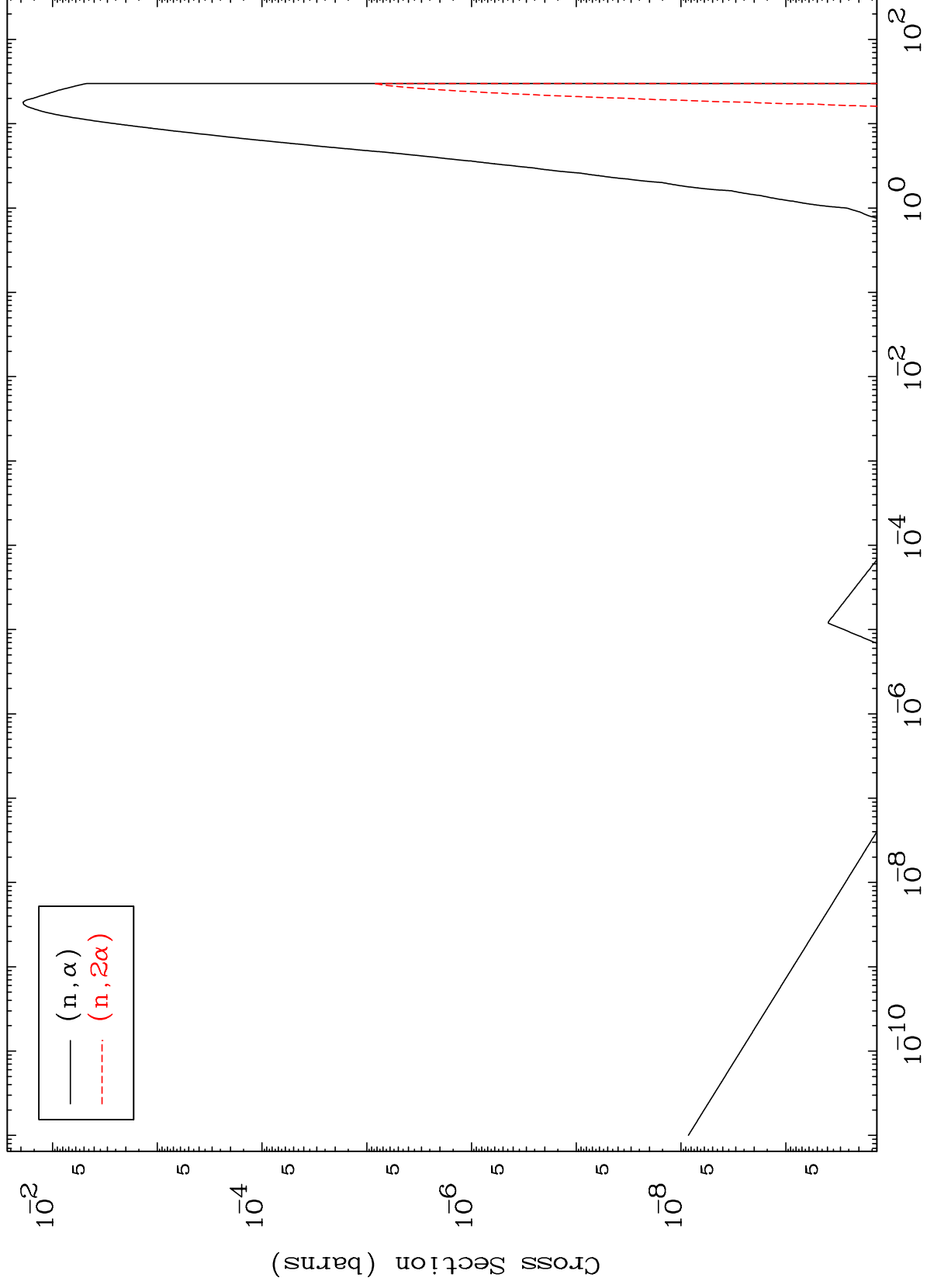
Incident Energy (MeV)

43-Tc-99m

MAT 4326

(n, $\alpha$ ) Levels  
293 Kelvin Cross Sections

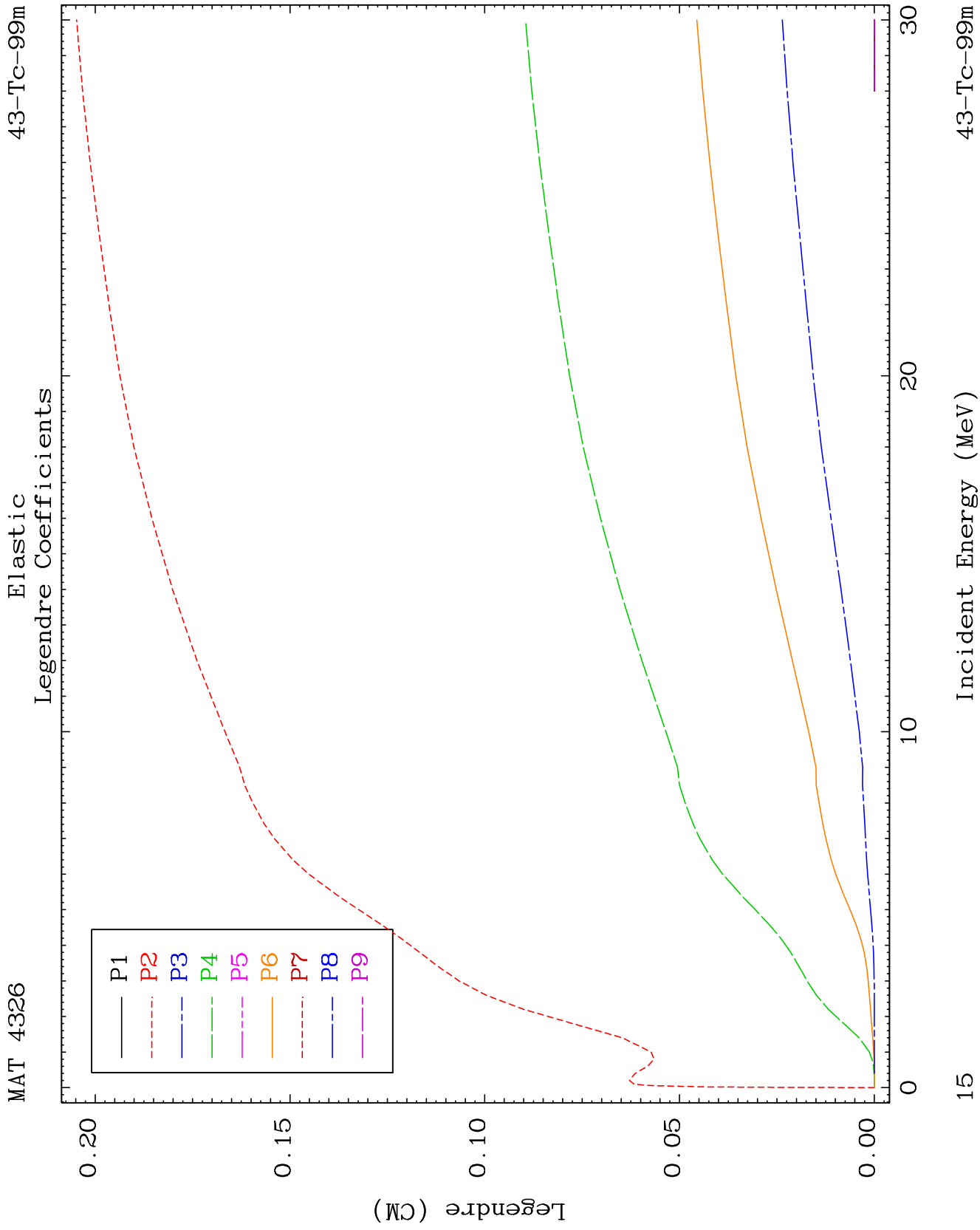
43-Tc-99m



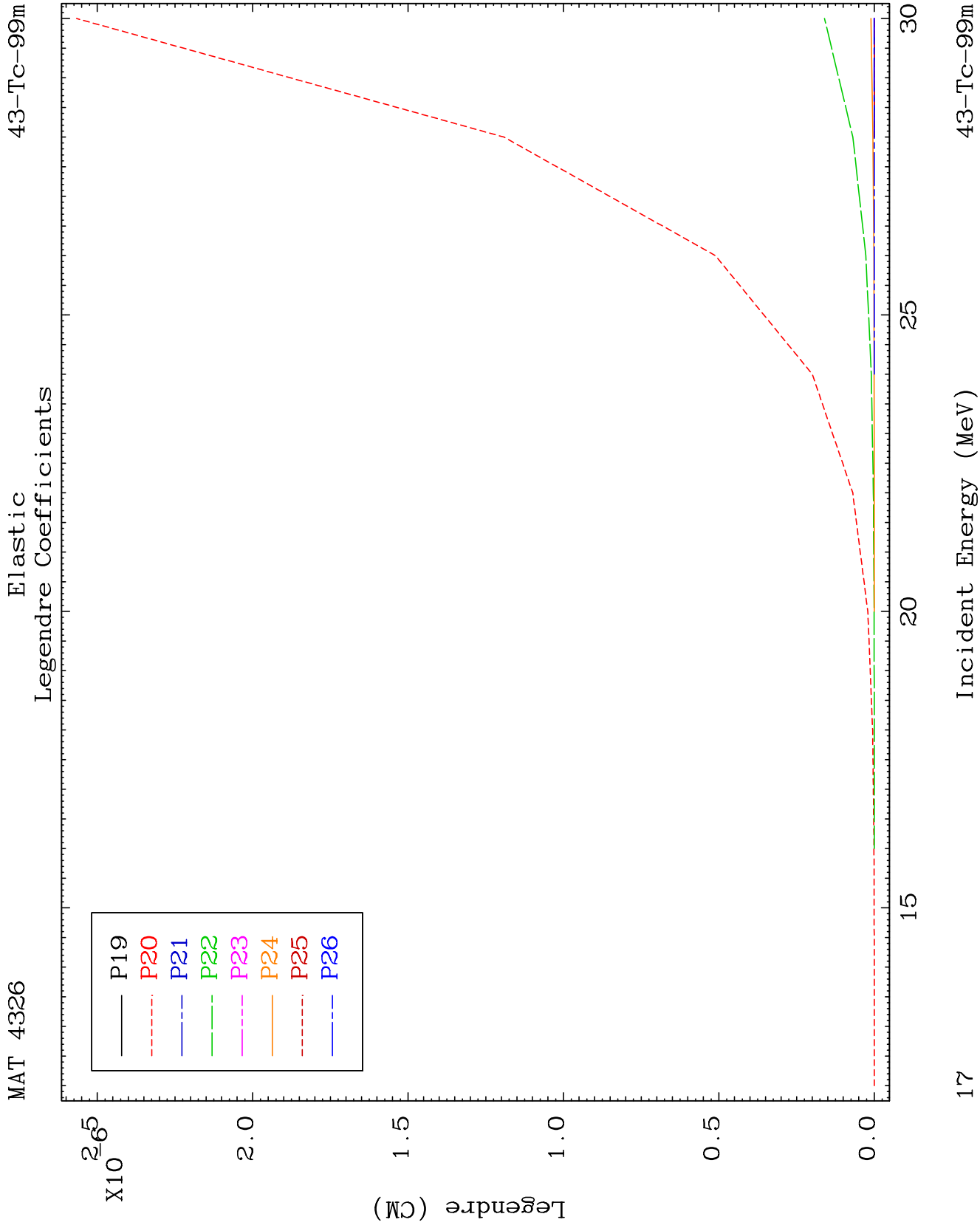
14

Incident Energy (MeV)

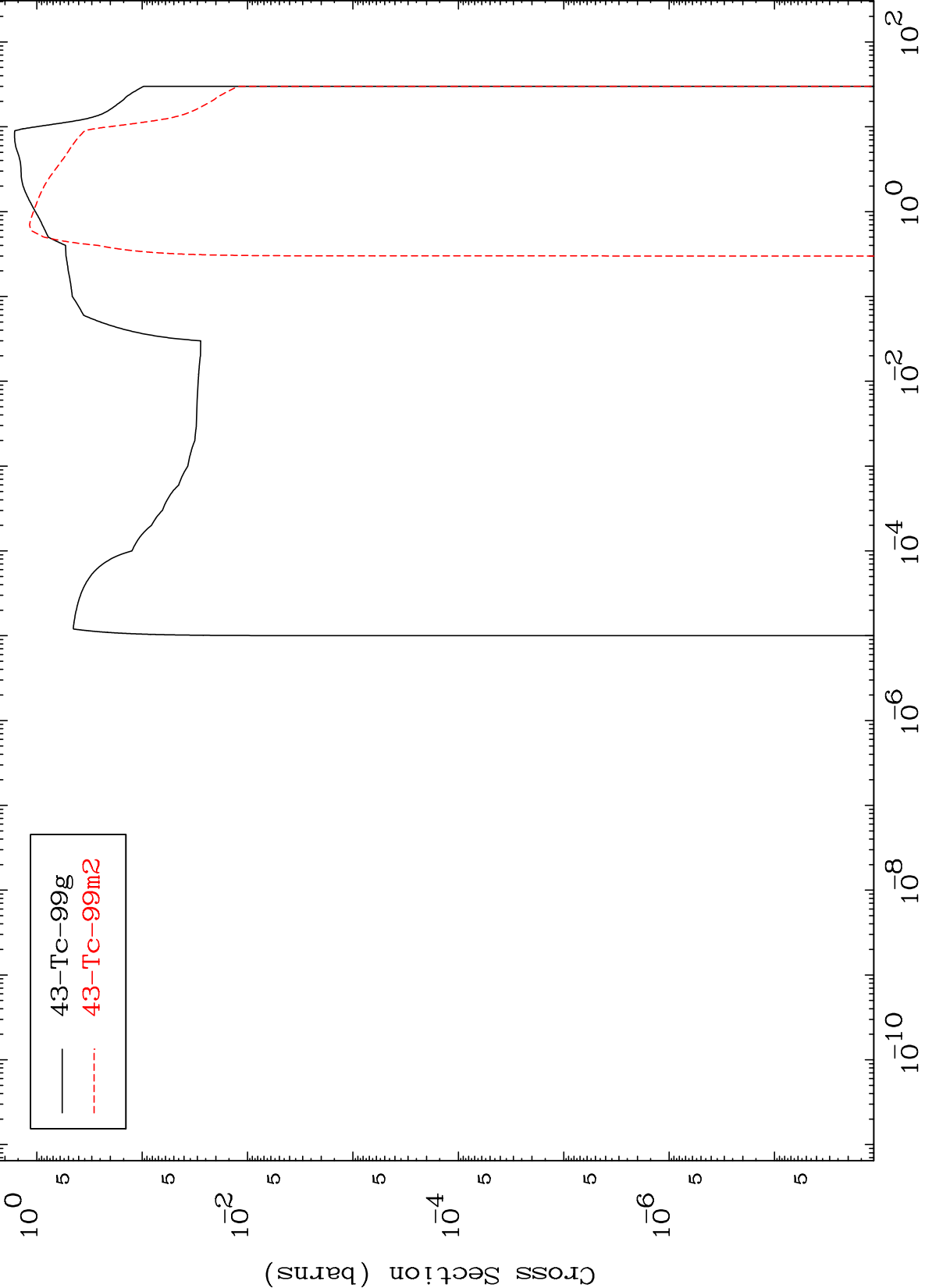
43-Tc-99m

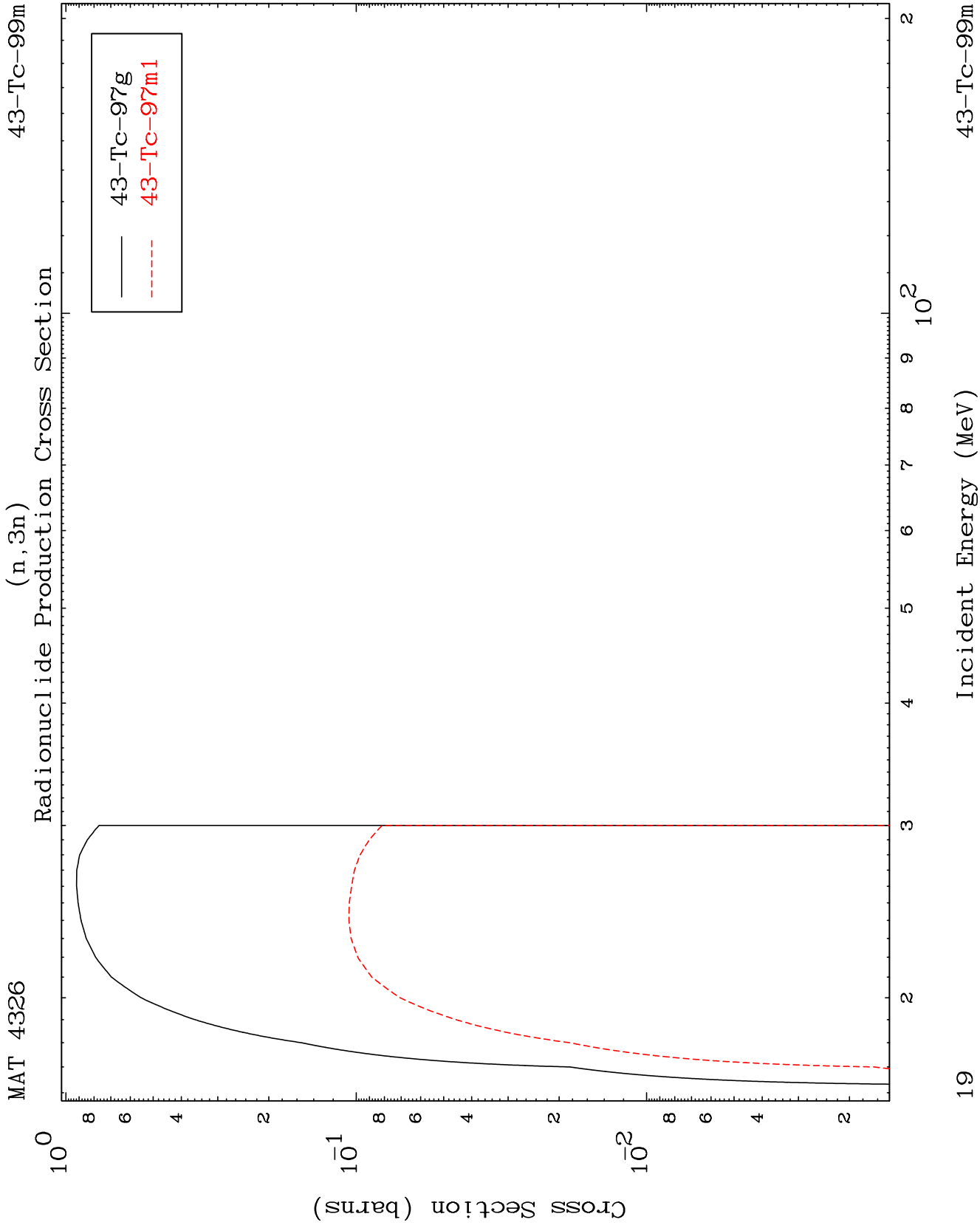






Inelastic  
Radionuclide Production Cross Section



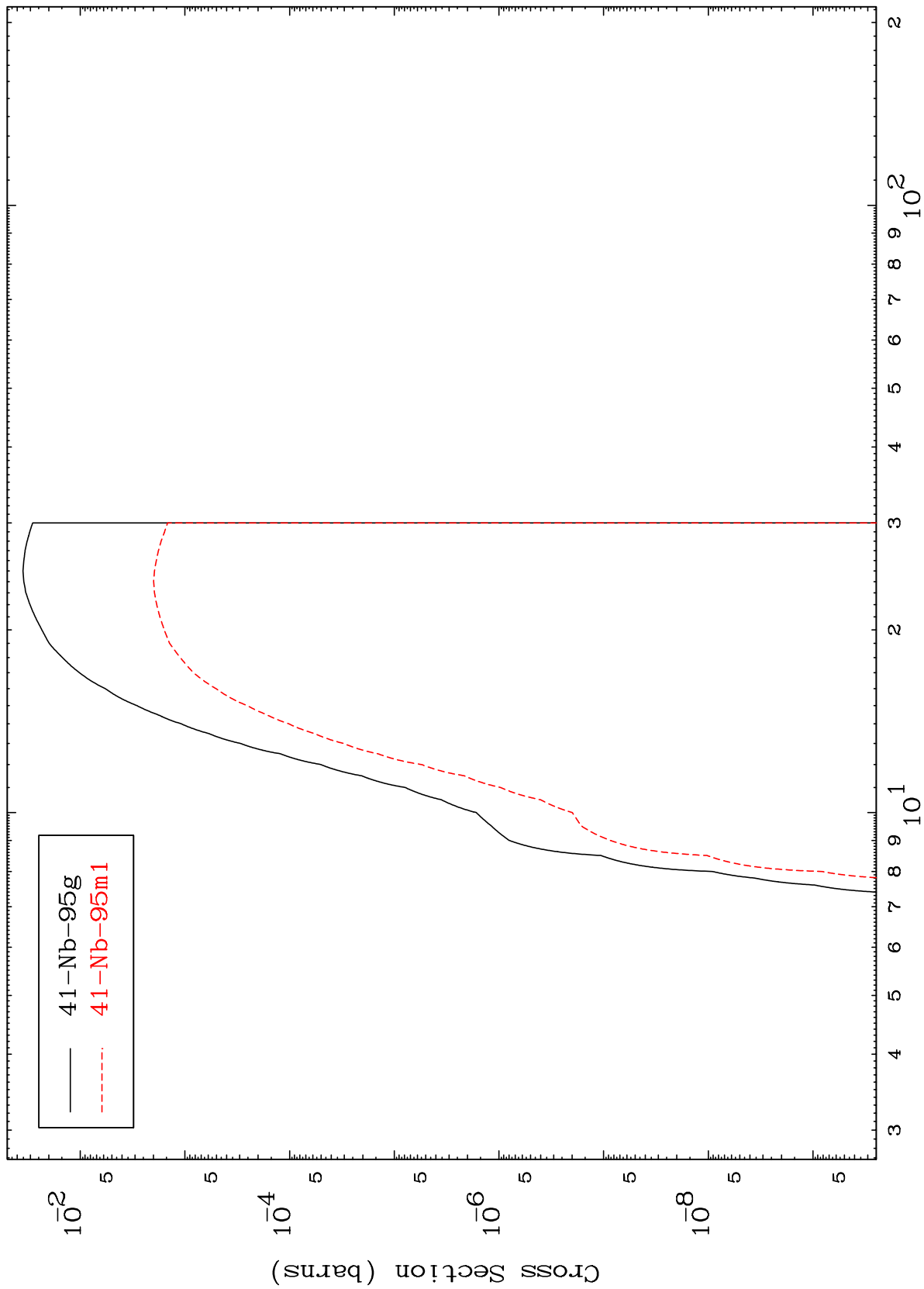


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43-Tc-99m

Radionuclide Production Cross Section

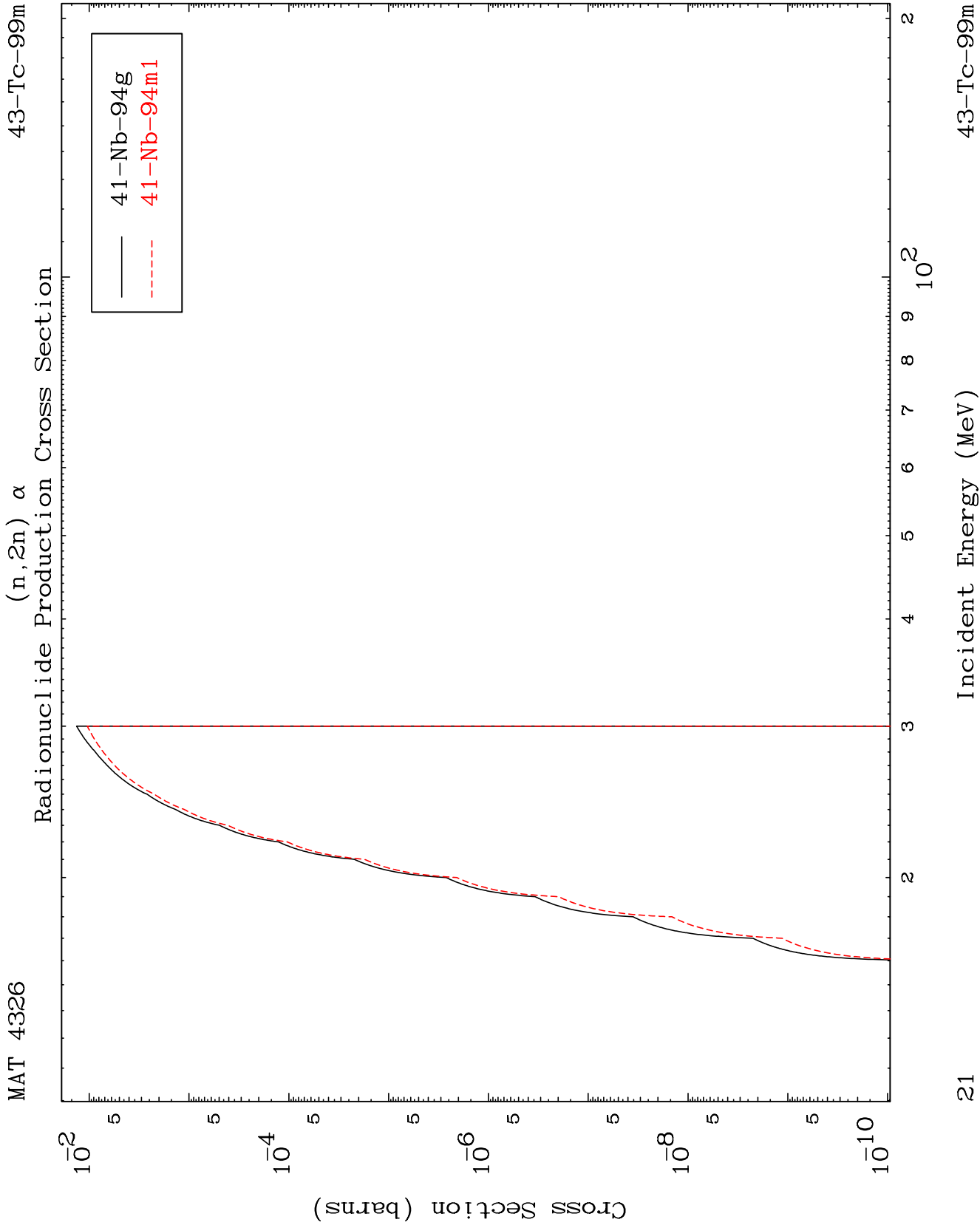
(n,n')  $\alpha$

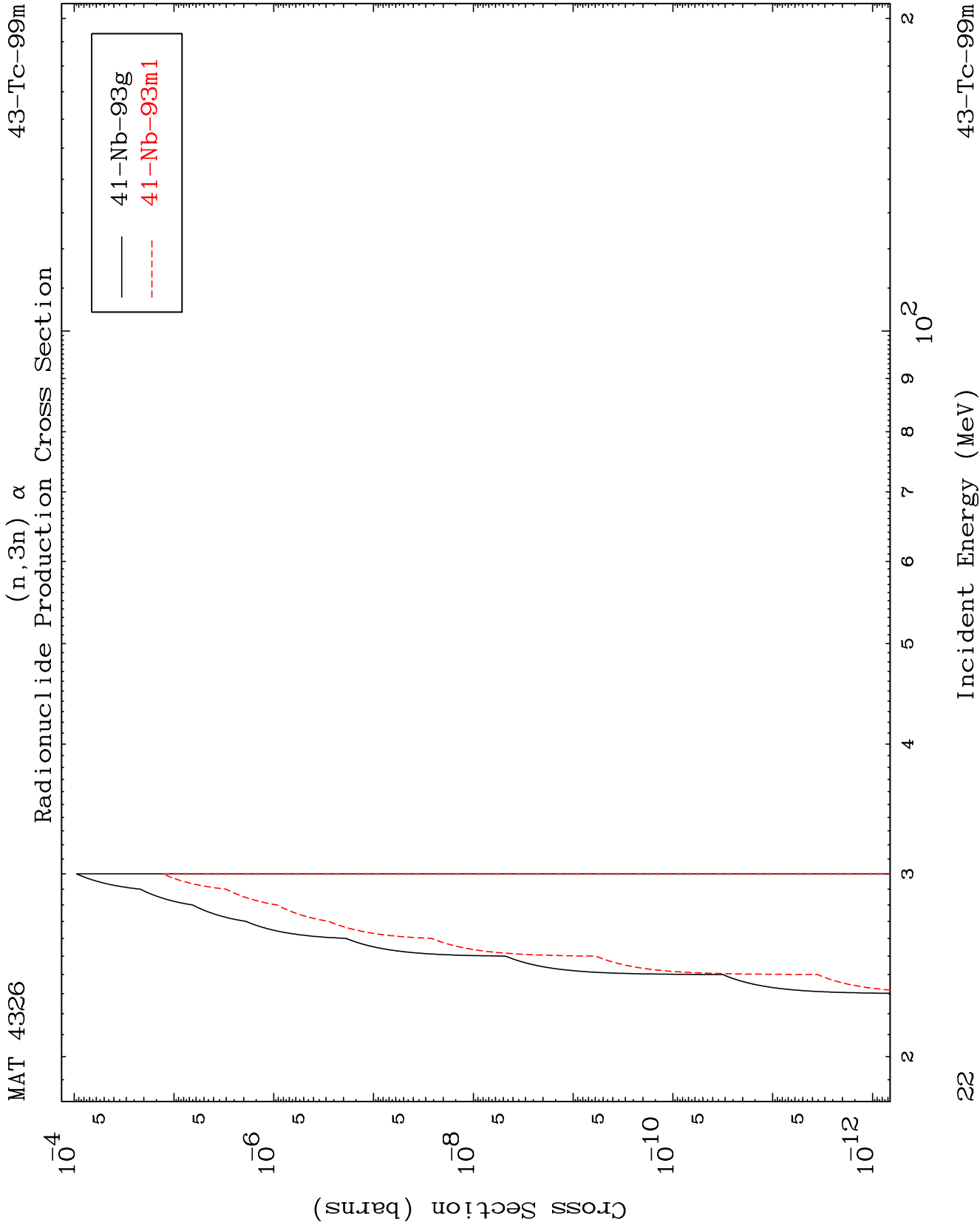


43-Tc-99m

Incident Energy (MeV)

20

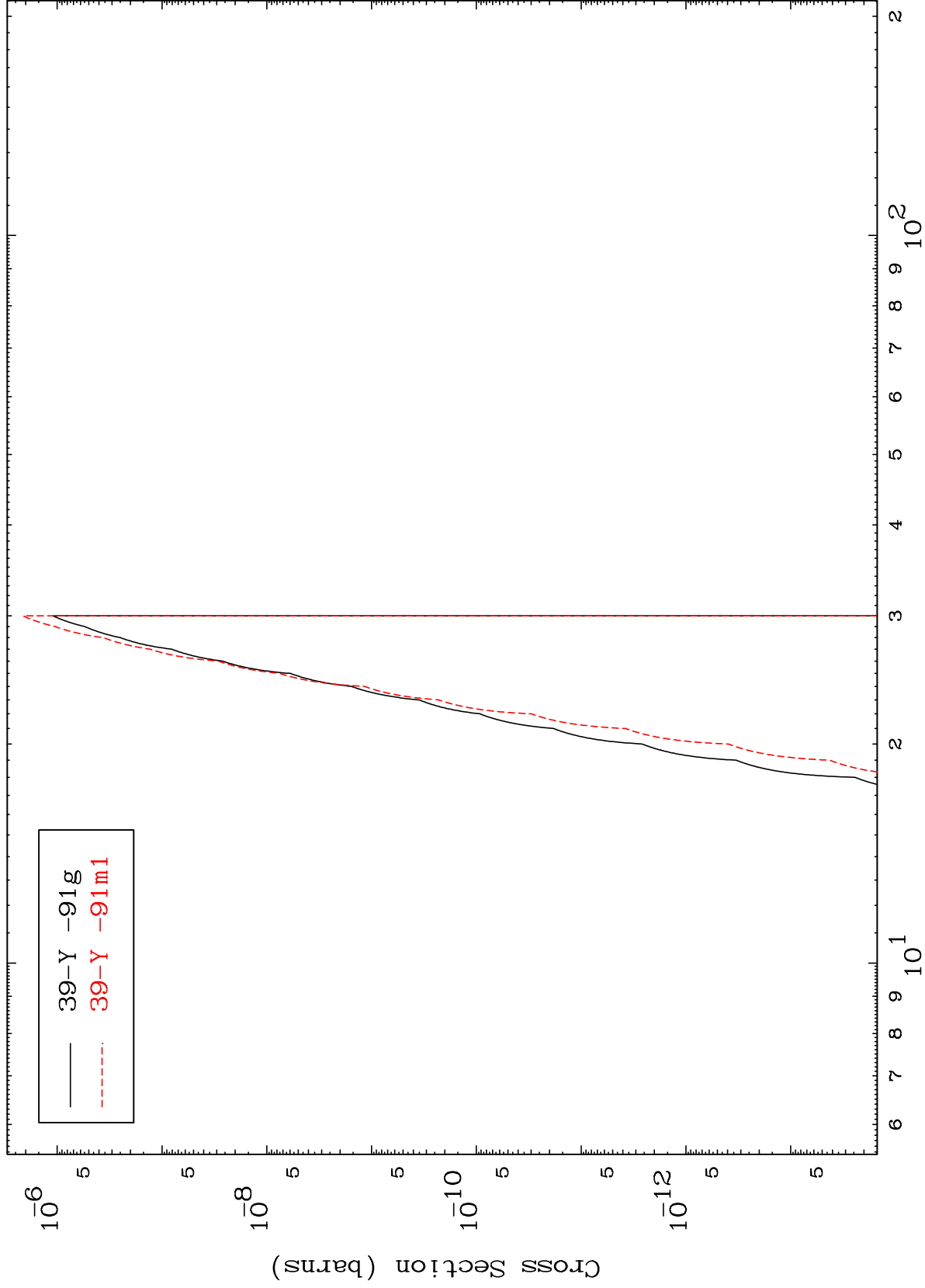




MAT 4326

43-Tc-99m

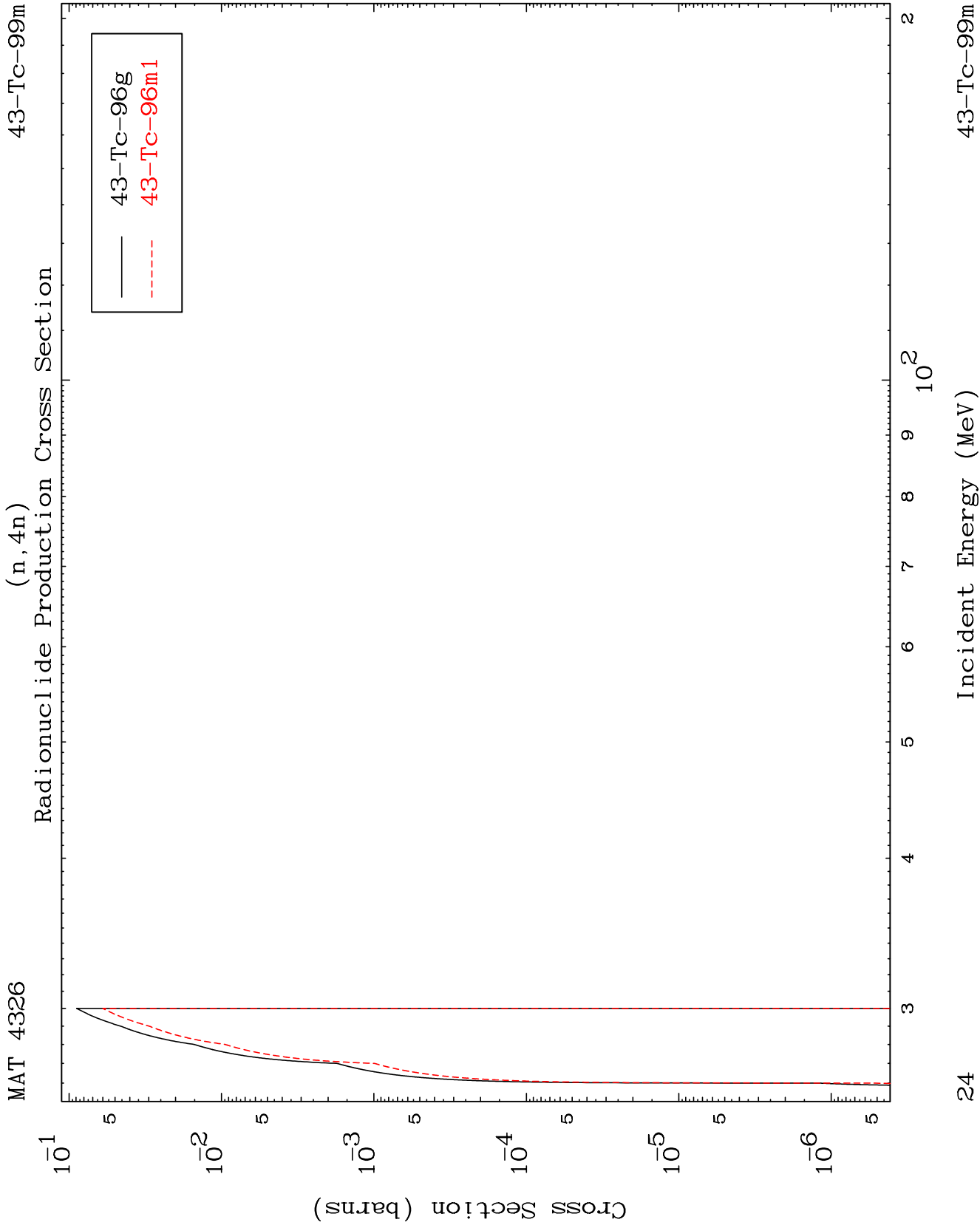
(n,n') 2 $\alpha$   
Radionuclide Production Cross Section



23

Incident Energy (MeV)

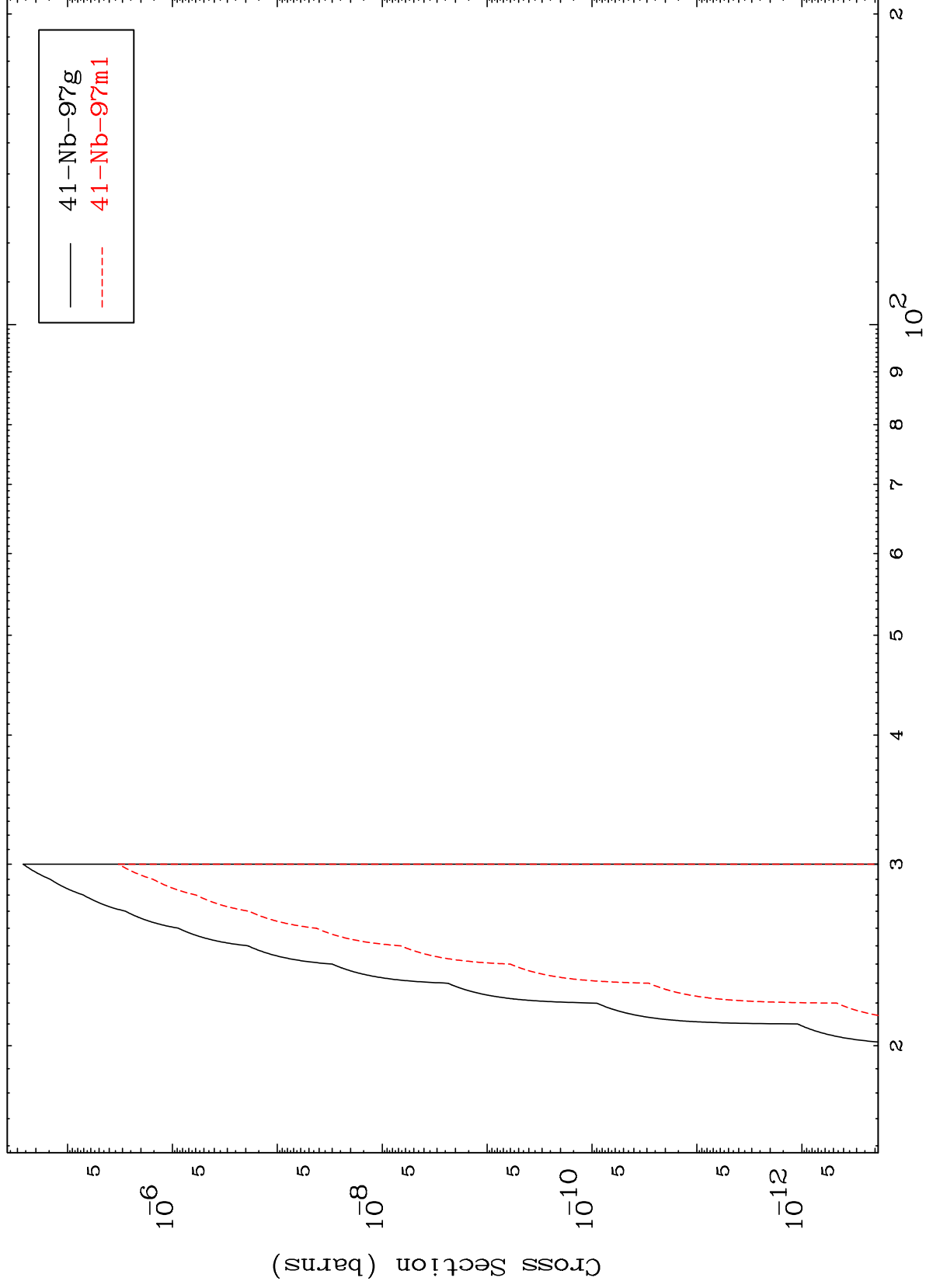
43-Tc-99m



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43-Tc-99m

(n,2n) p  
Radionuclide Production Cross Section



25

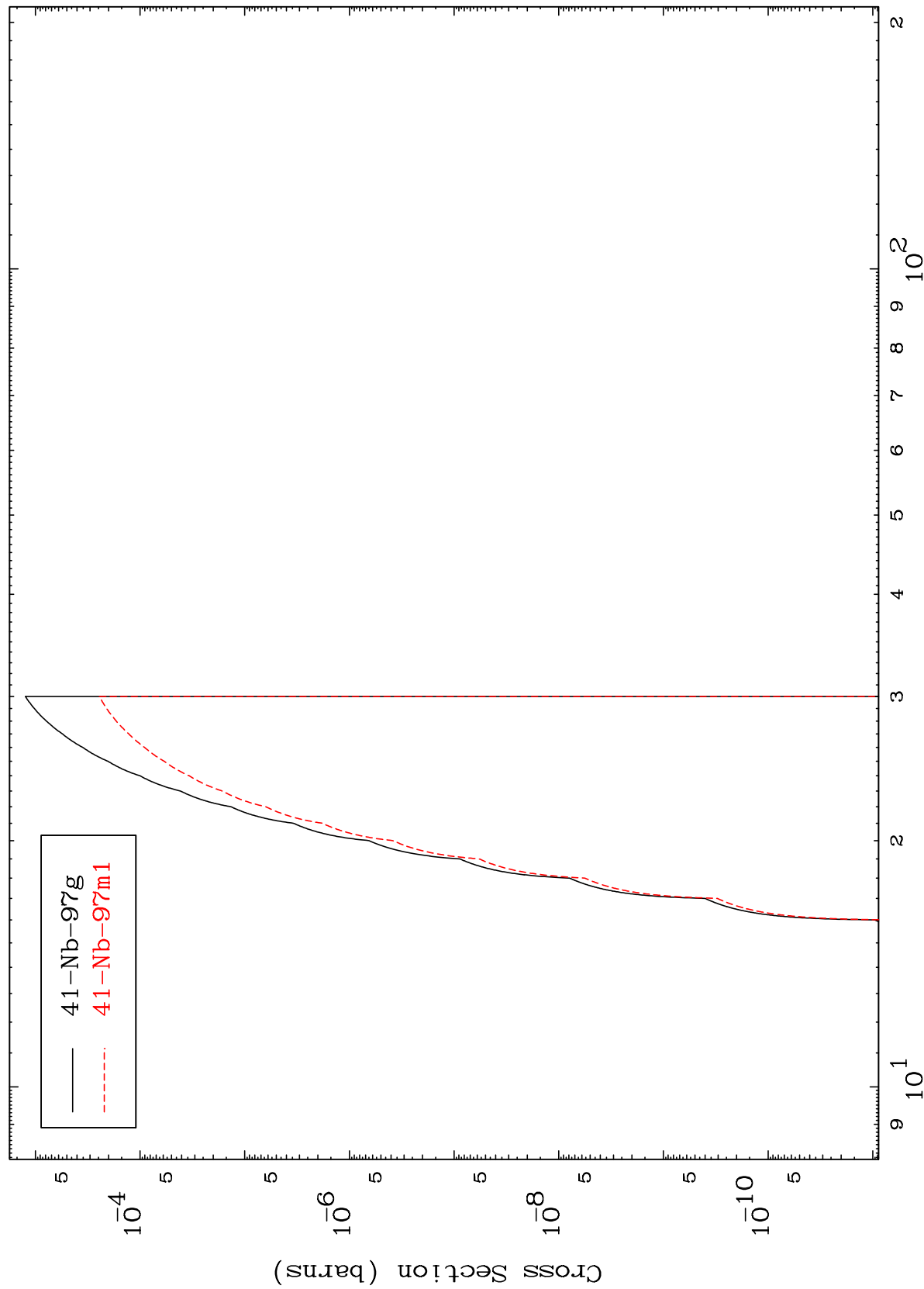
Incident Energy (MeV)

43-Tc-99m

MAT 4326

43-Tc-99m

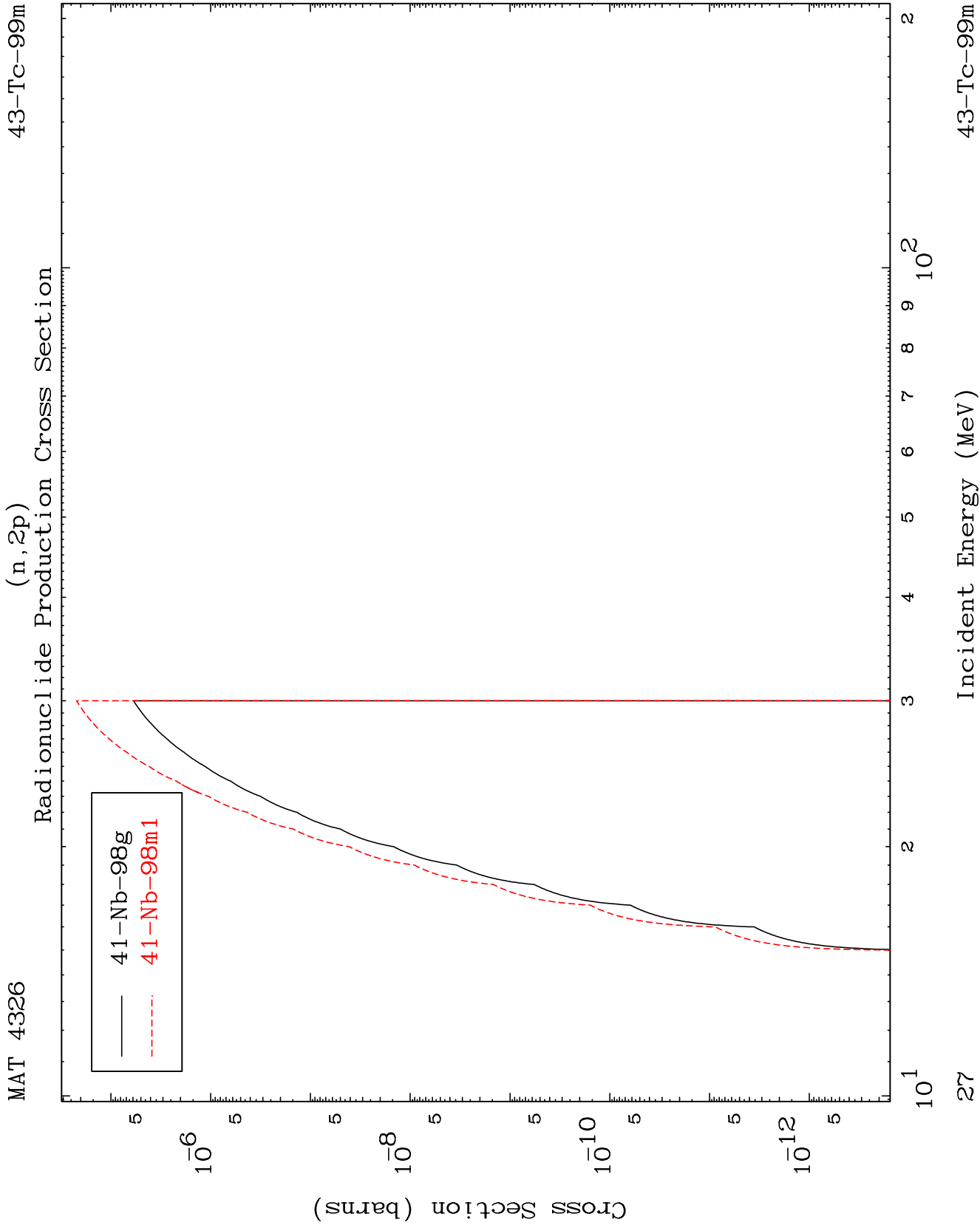
Radionuclide Production Cross Section  
(n,He-3)



43-Tc-99m

Incident Energy (MeV)

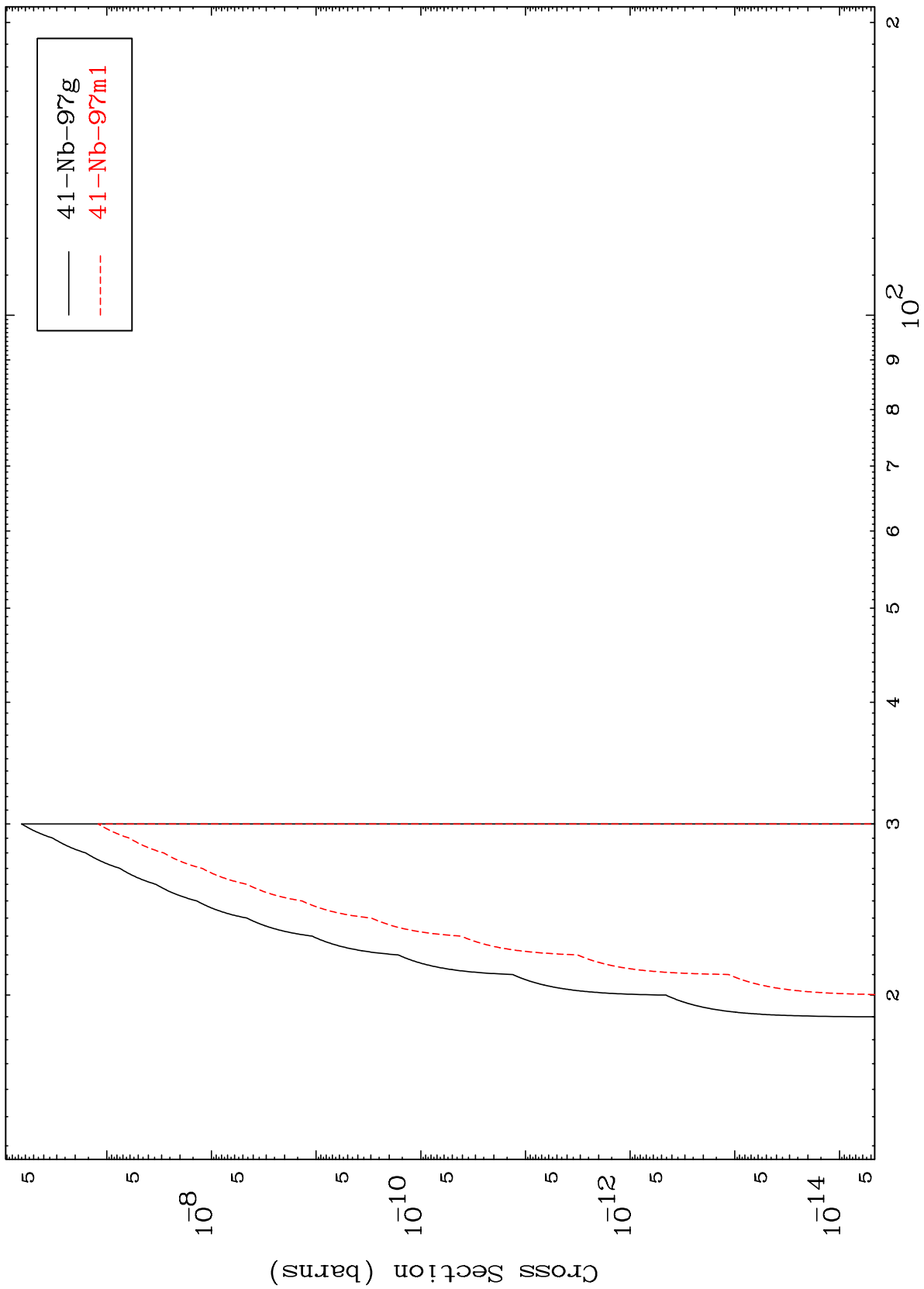
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MAT 4326

43-Tc-99m

(n,p) d  
Radionuclide Production Cross Section



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Incident Energy (MeV)

43-Tc-99m