

Program EVALPLOT
(Version 2021-1)

by

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(Present Contact Information)

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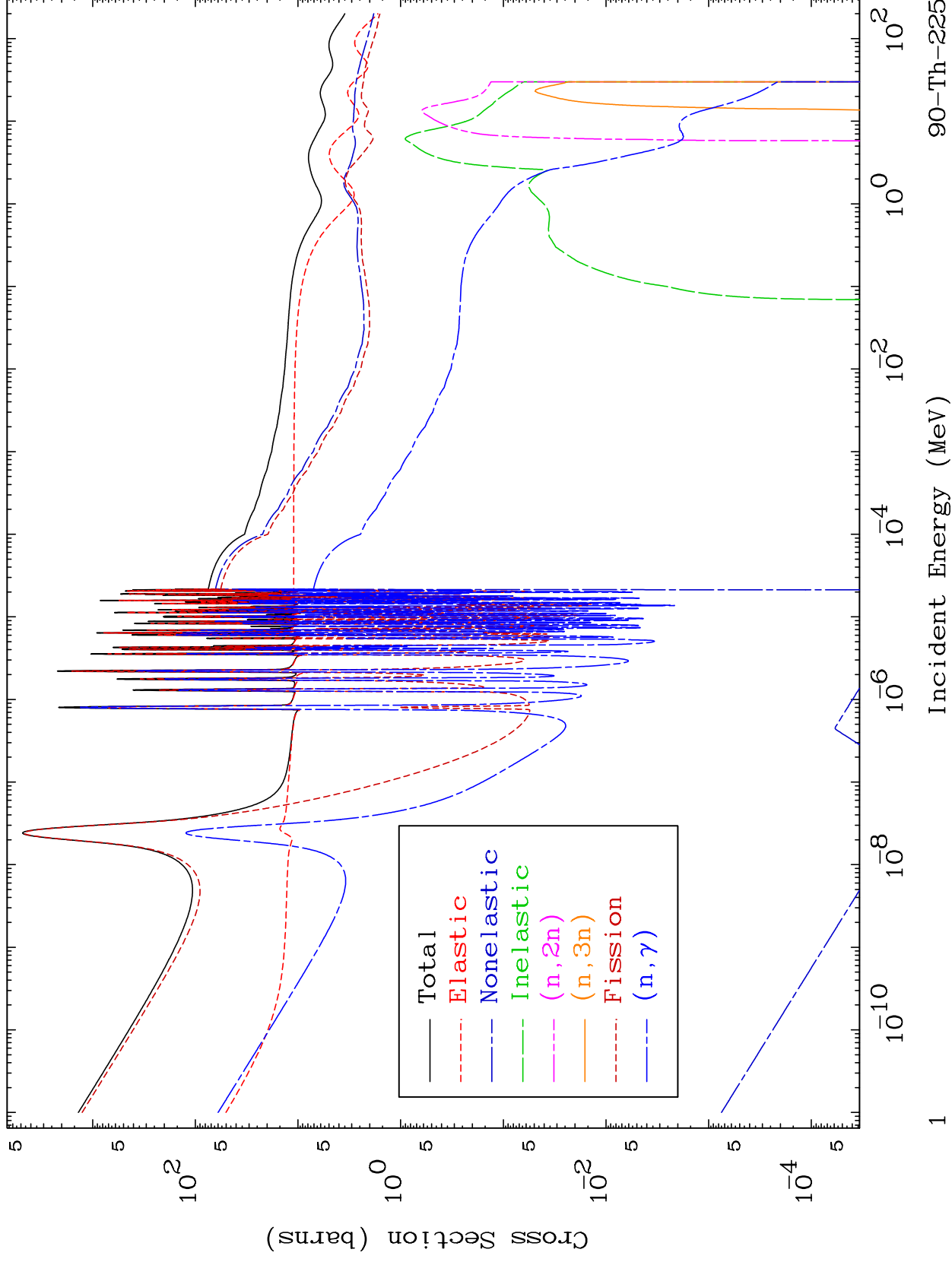
E.Mail:redcullen1@comcast.net
Web:redcullen1.net/HOMEPAGE.NEW

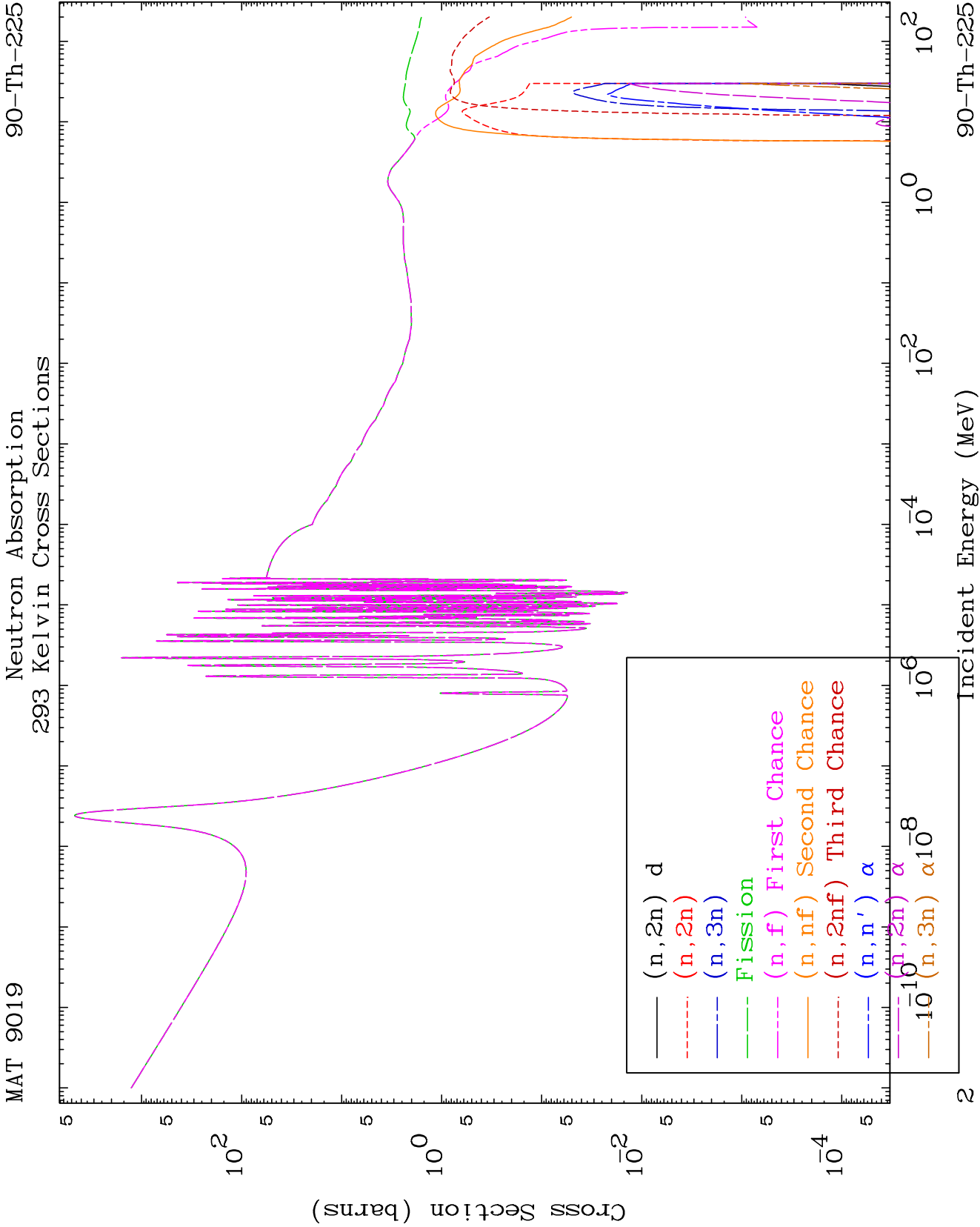
Press Mouse Button to Start

MAT 9019

Neutron Major
293 Kelvin Cross Sections

90-Th-225

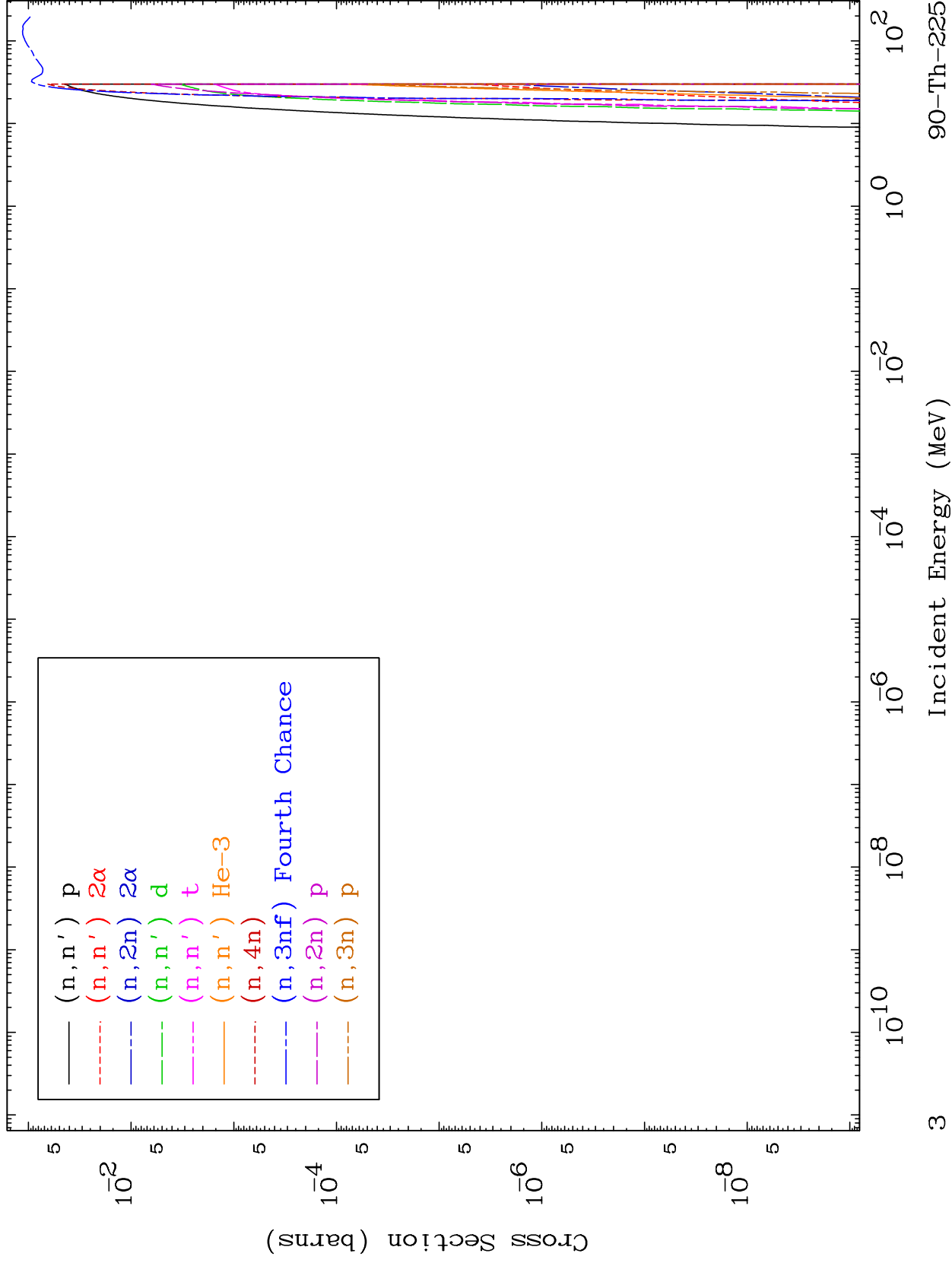


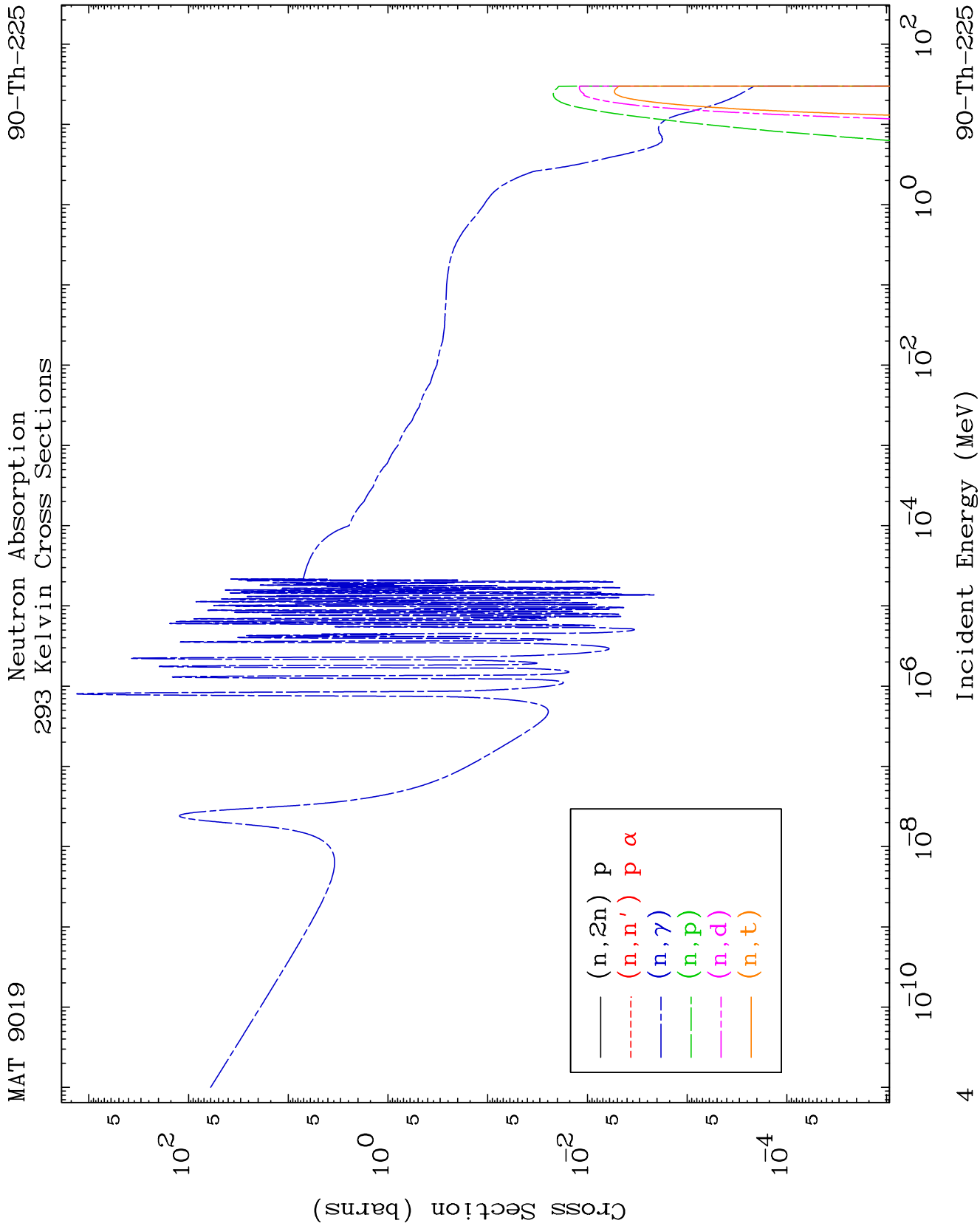


MAT 9019

Neutron Absorption
293 Kelvin Cross Sections

90-Th-225

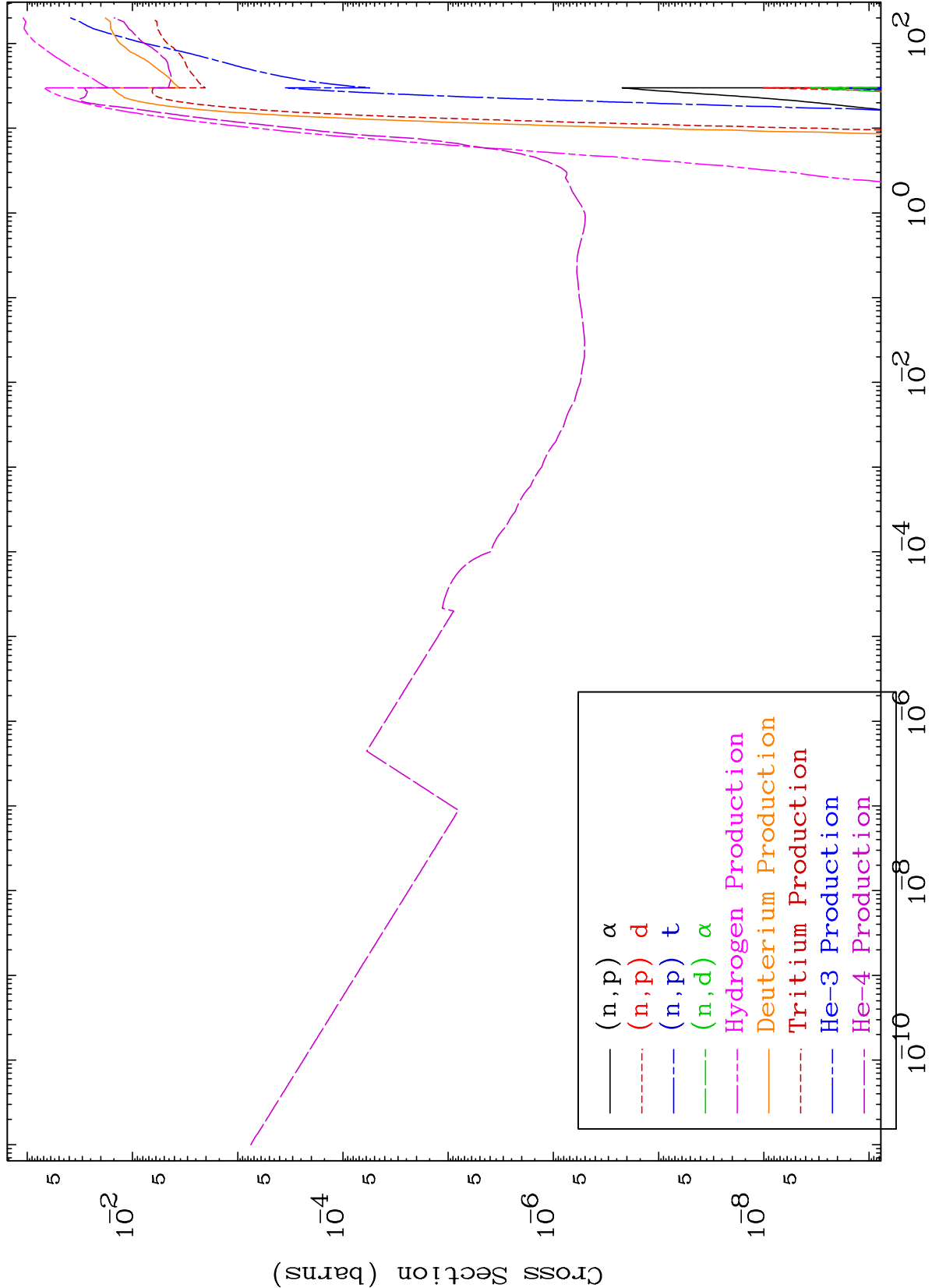


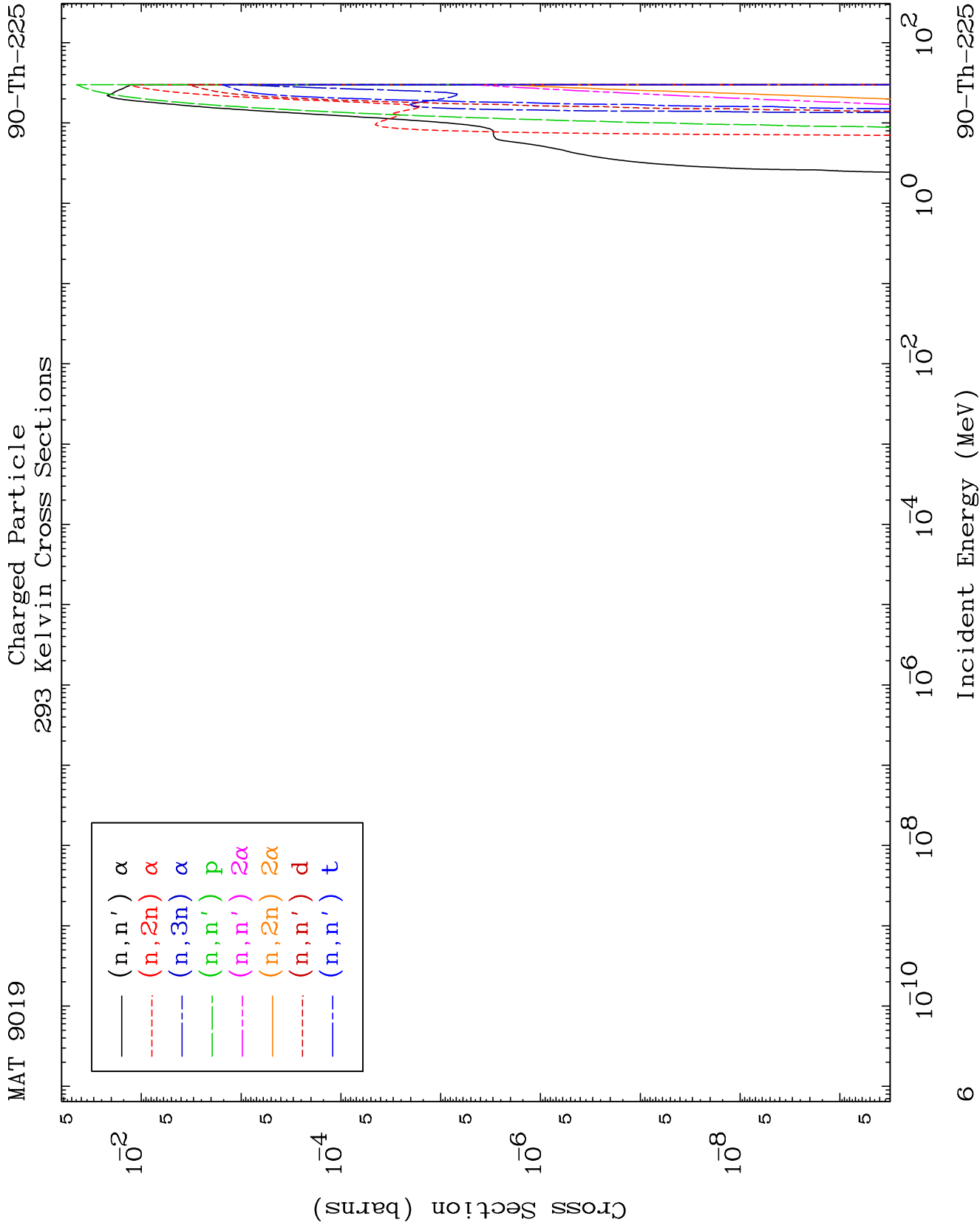


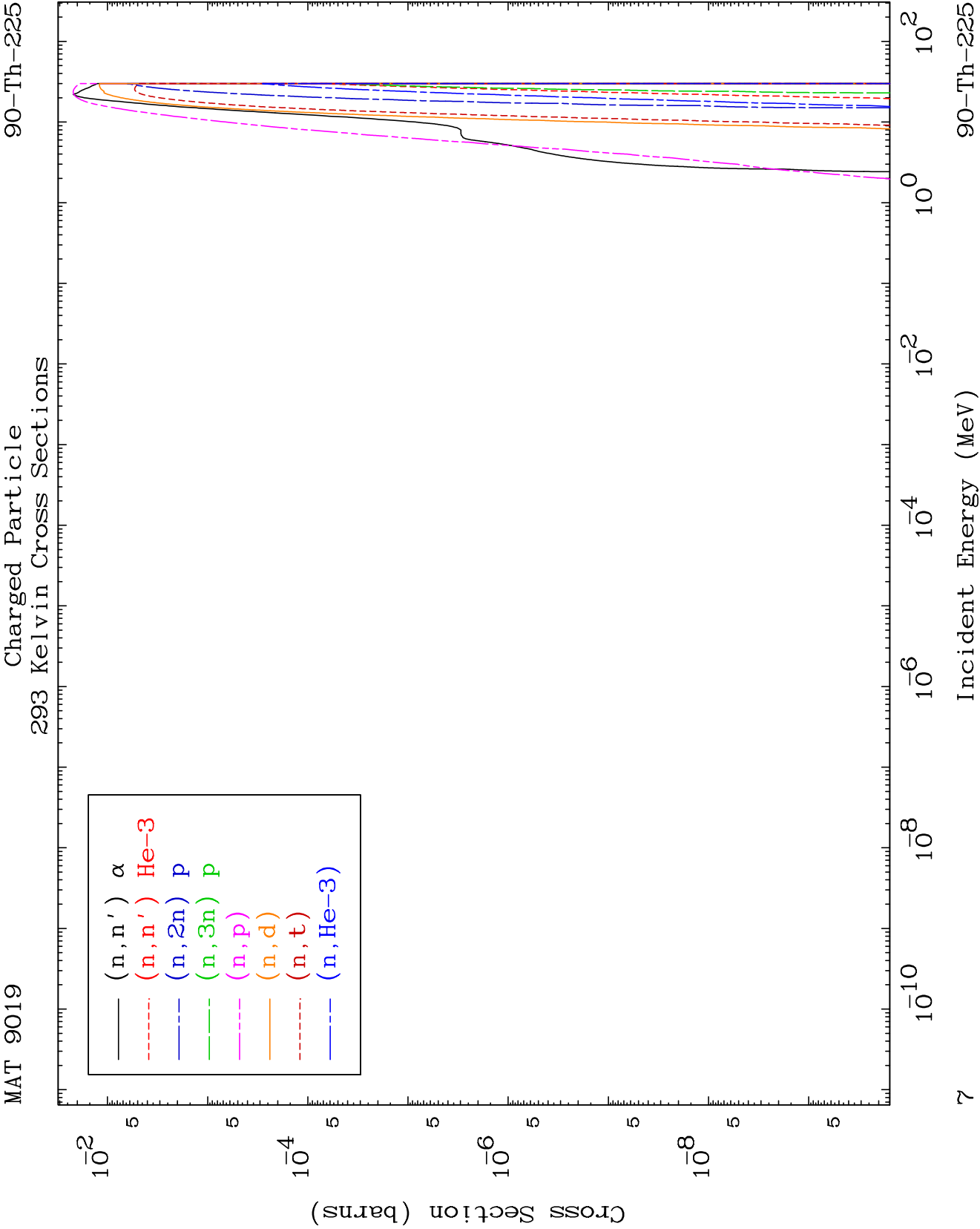
MAT 9019

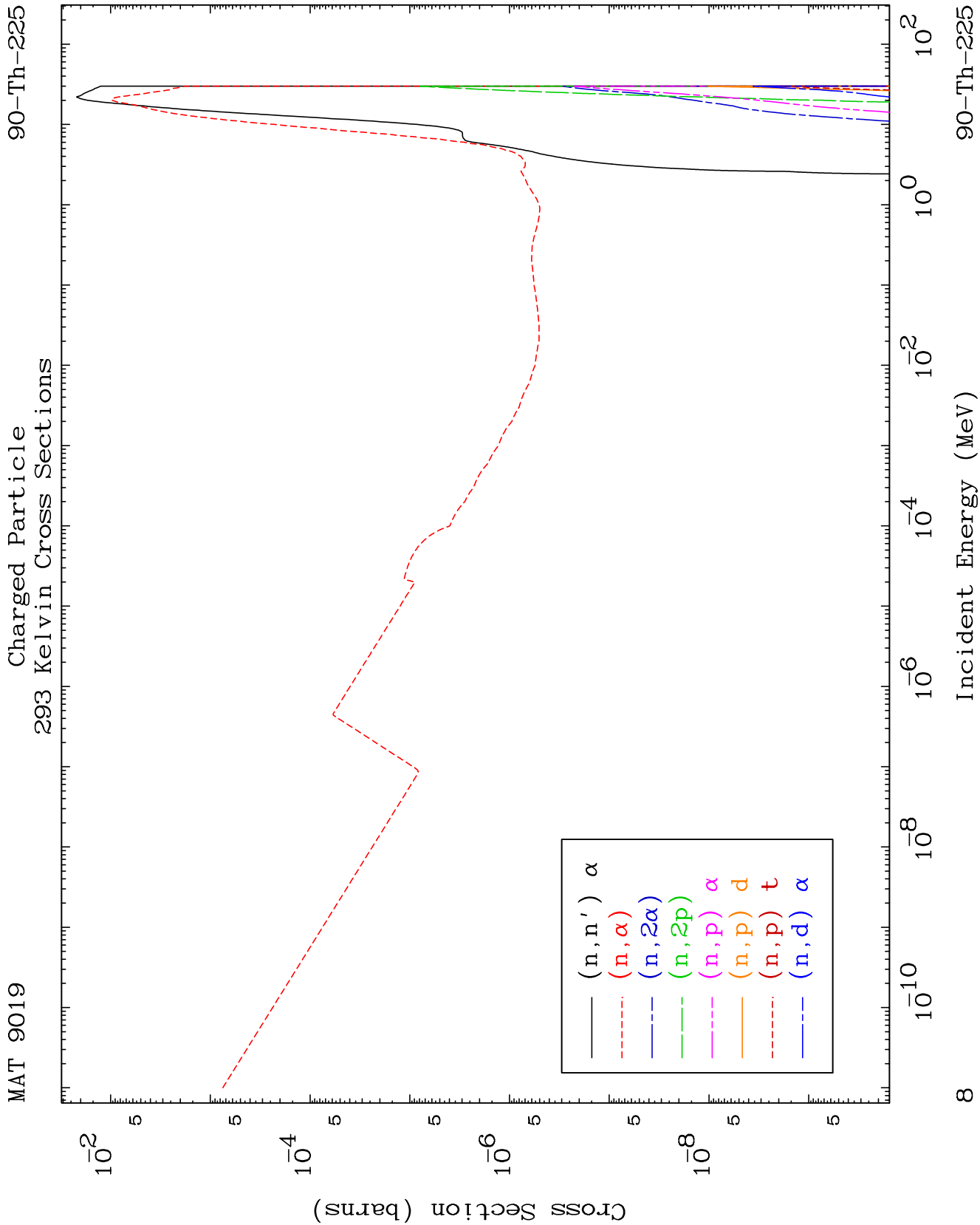
Neutron Absorption
293 Kelvin Cross Sections

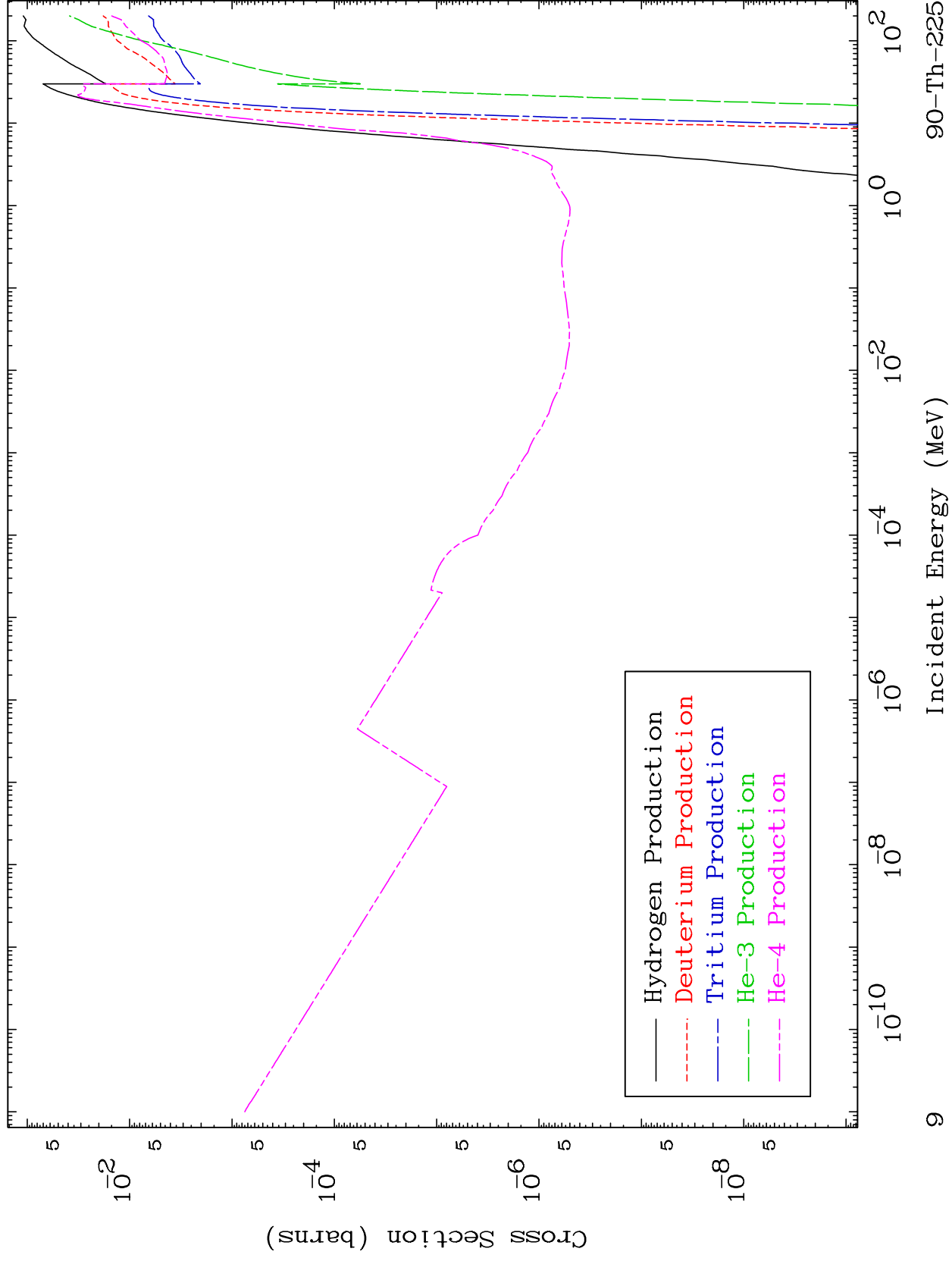
90-Th-225

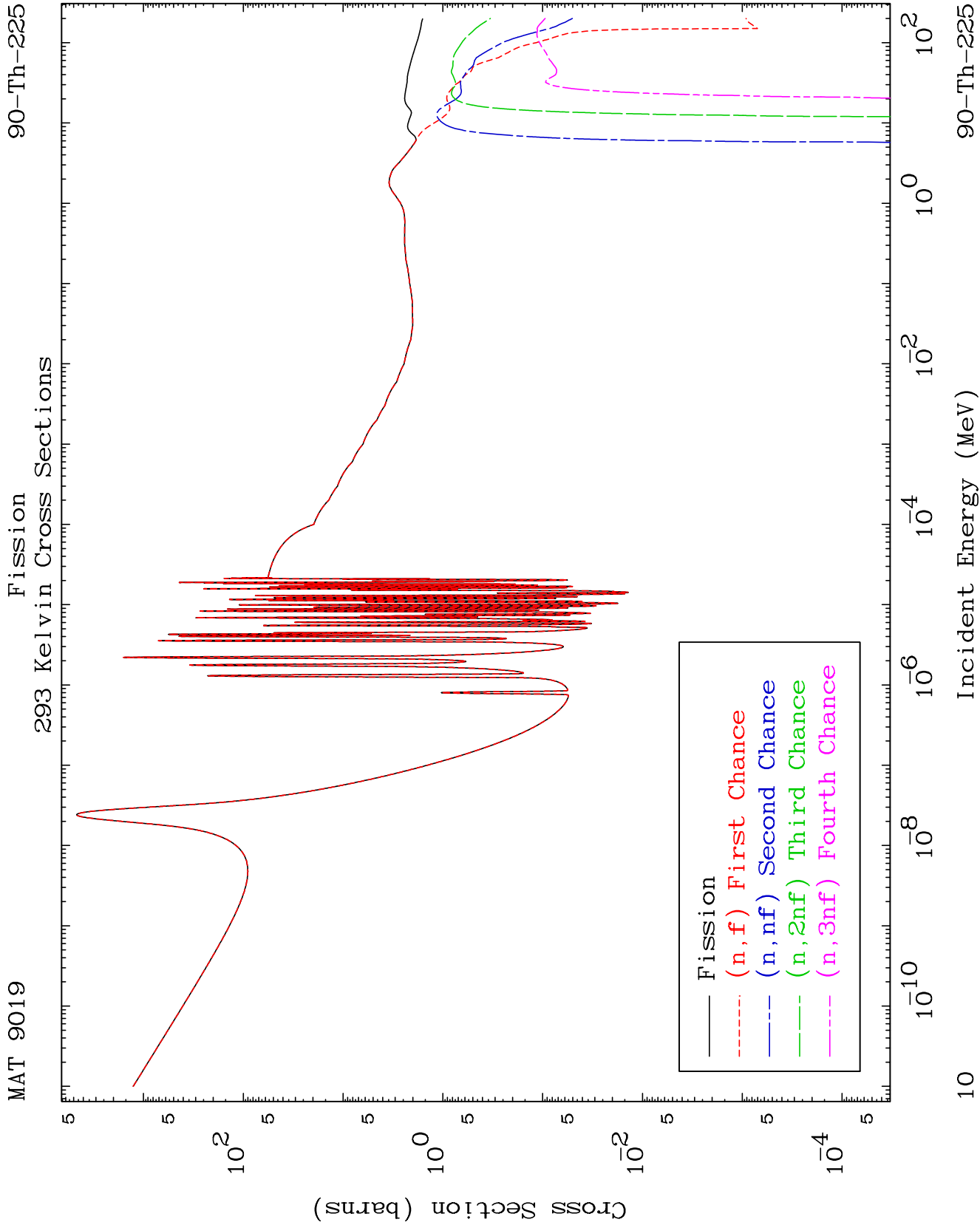


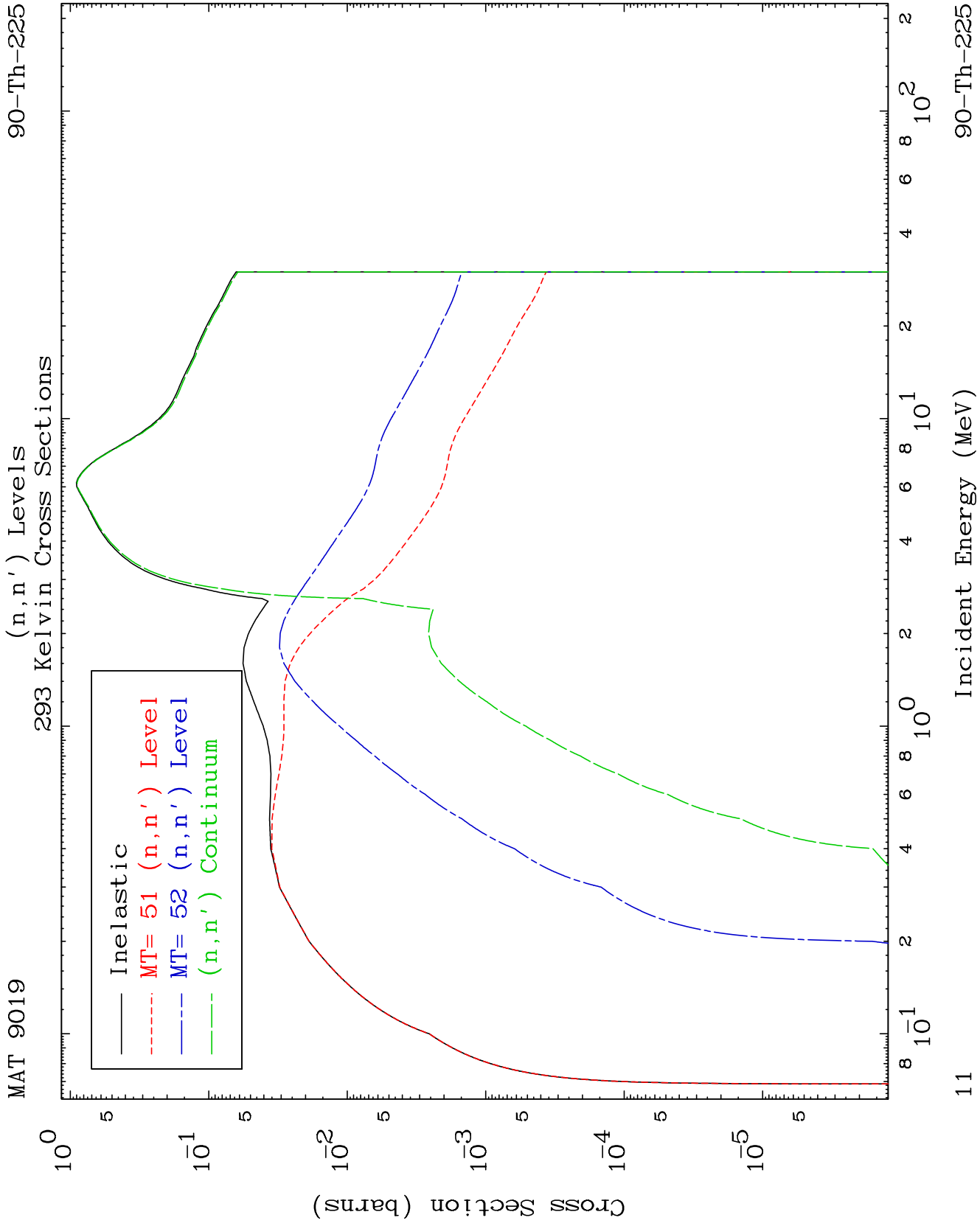








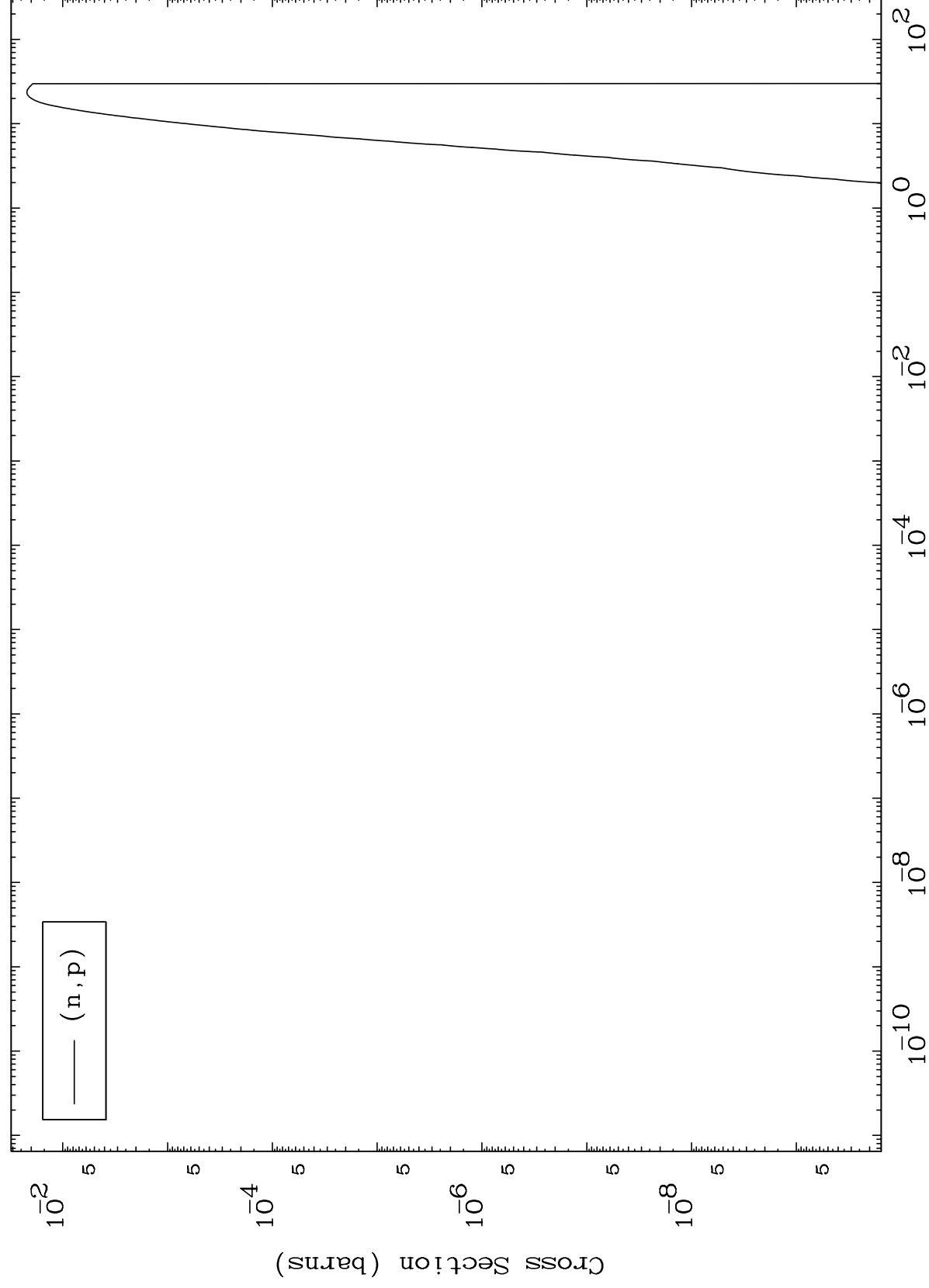




MAT 9019

(n,p) Levels
293 Kelvin Cross Sections

90-Th-225



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Incident Energy (MeV)

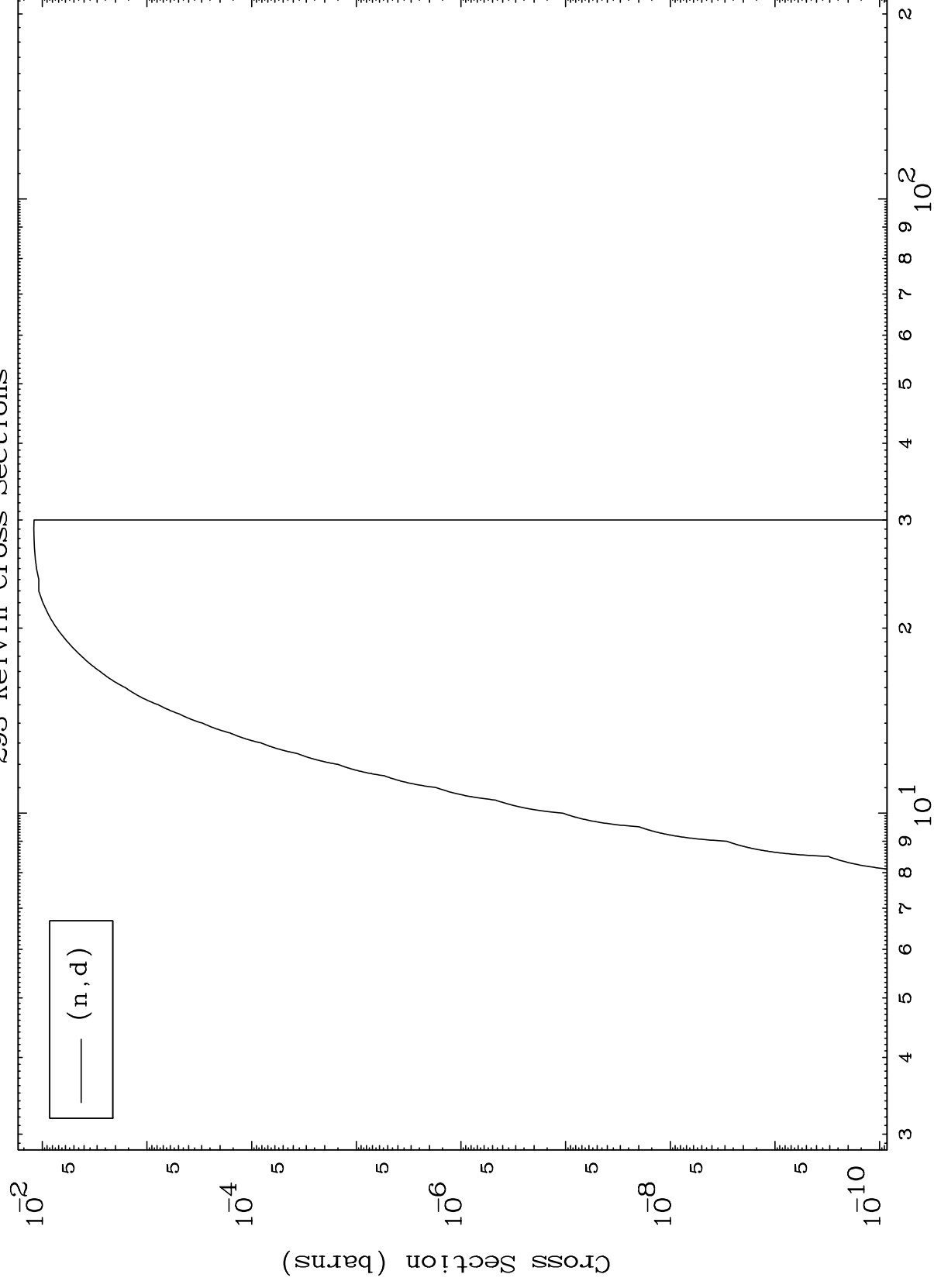
90-Th-225

MAT 9019

(n, d) Levels

90-Th-225

293 Kelvin Cross Sections



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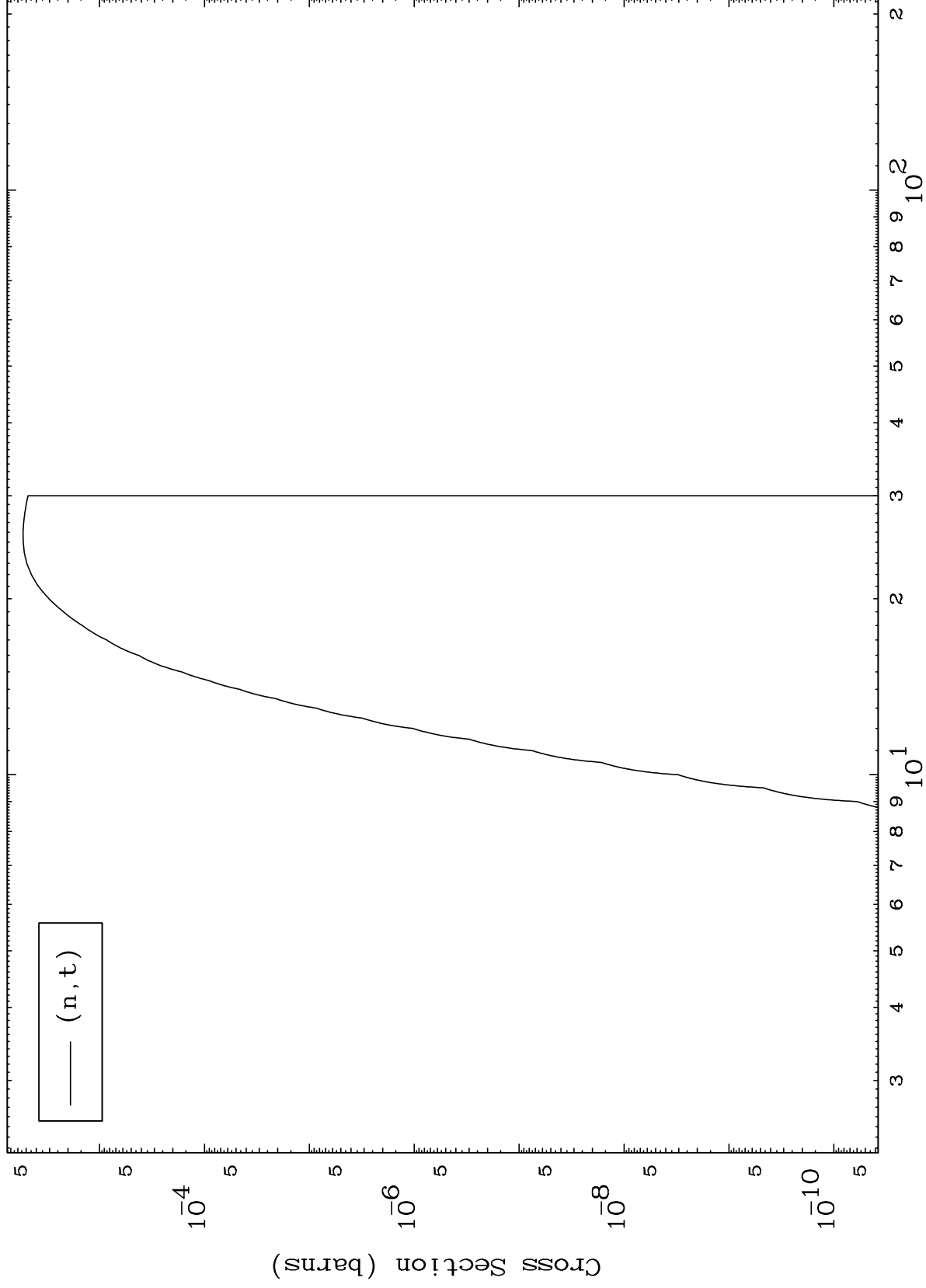
Incident Energy (MeV)

90-Th-225

MAT 9019

90-Th-225

(n,t) Levels
293 Kelvin Cross Sections



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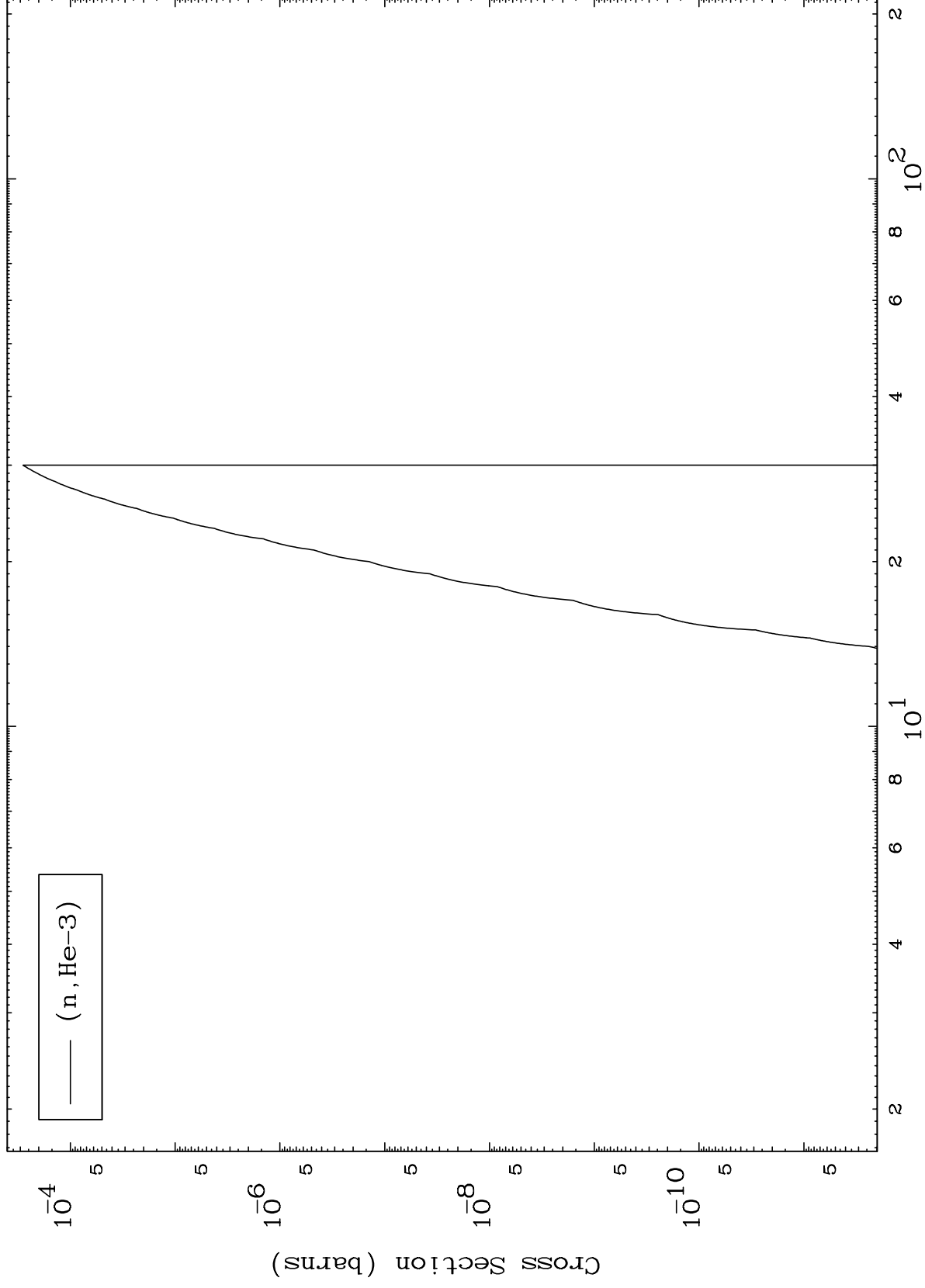
Incident Energy (MeV)

90-Th-225

MAT 9019

(n,He3) Levels
293 Kelvin Cross Sections

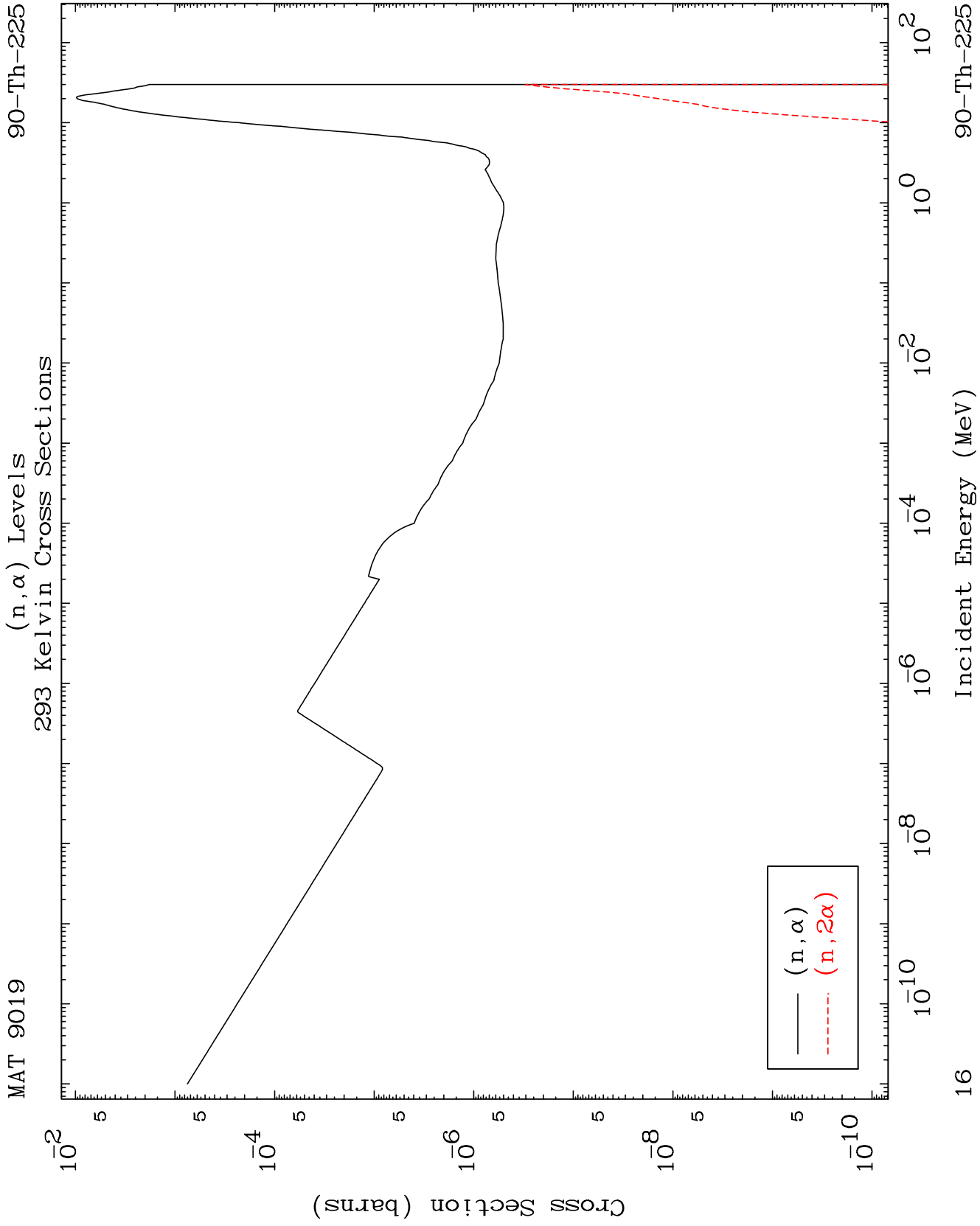
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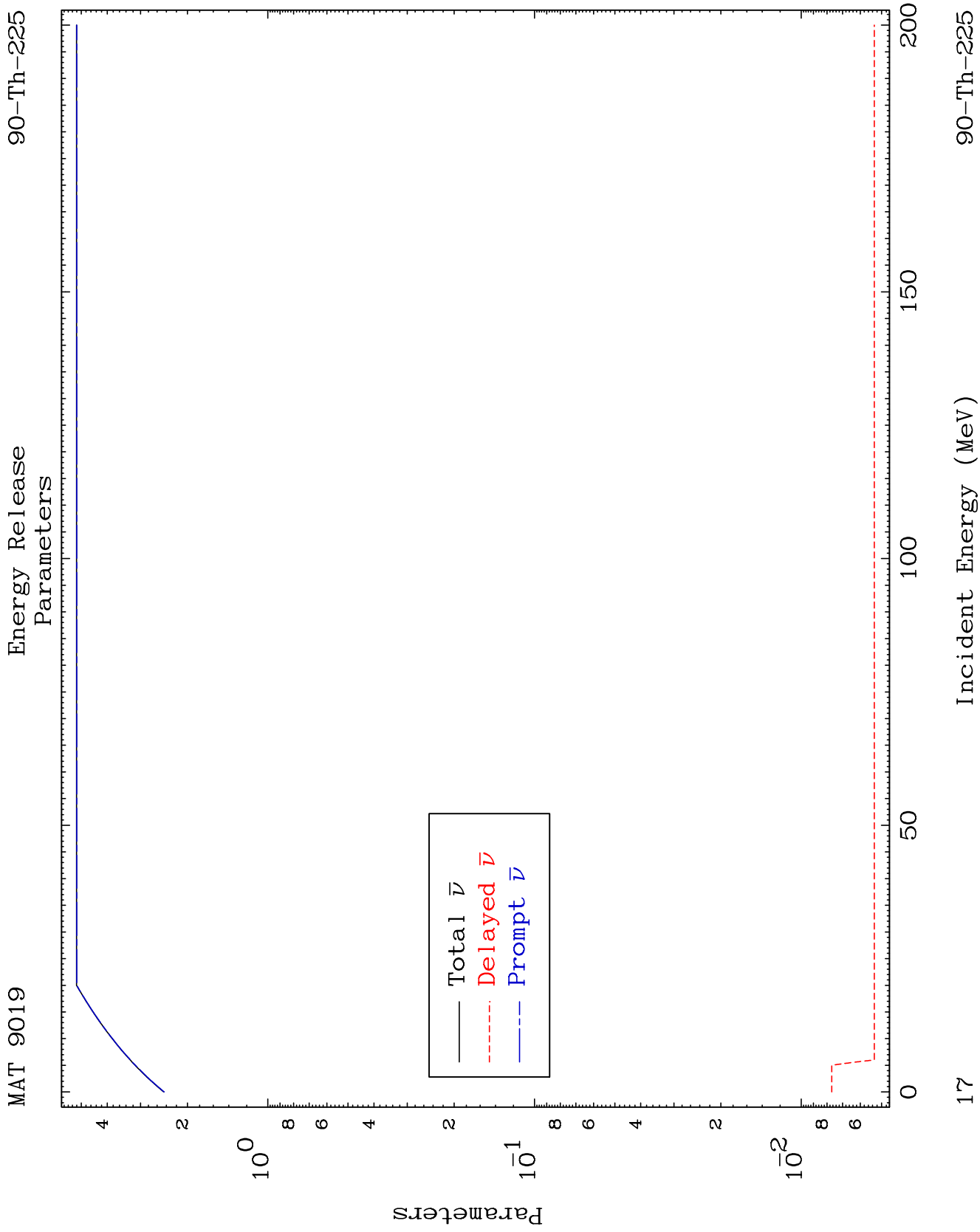


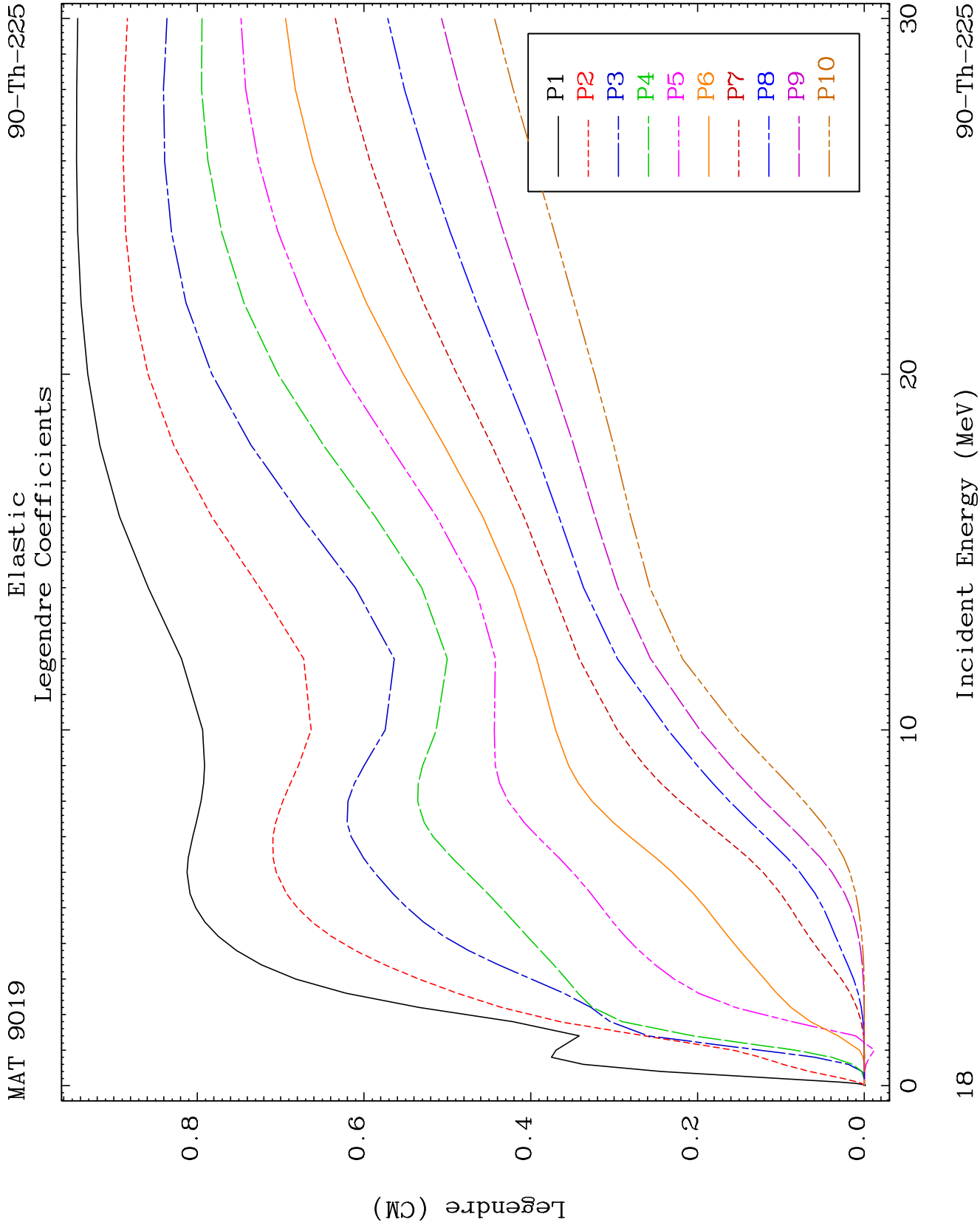
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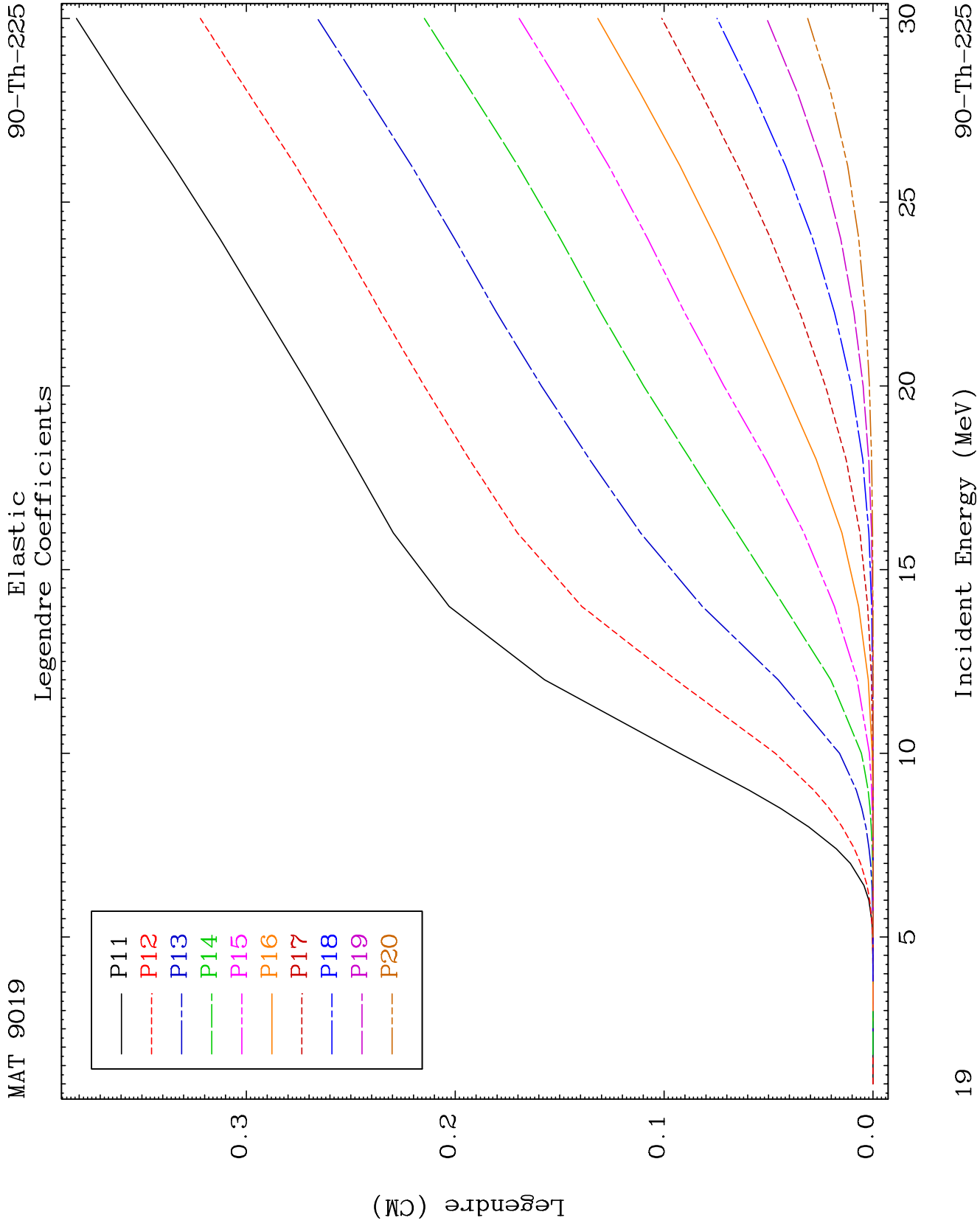
Incident Energy (MeV)

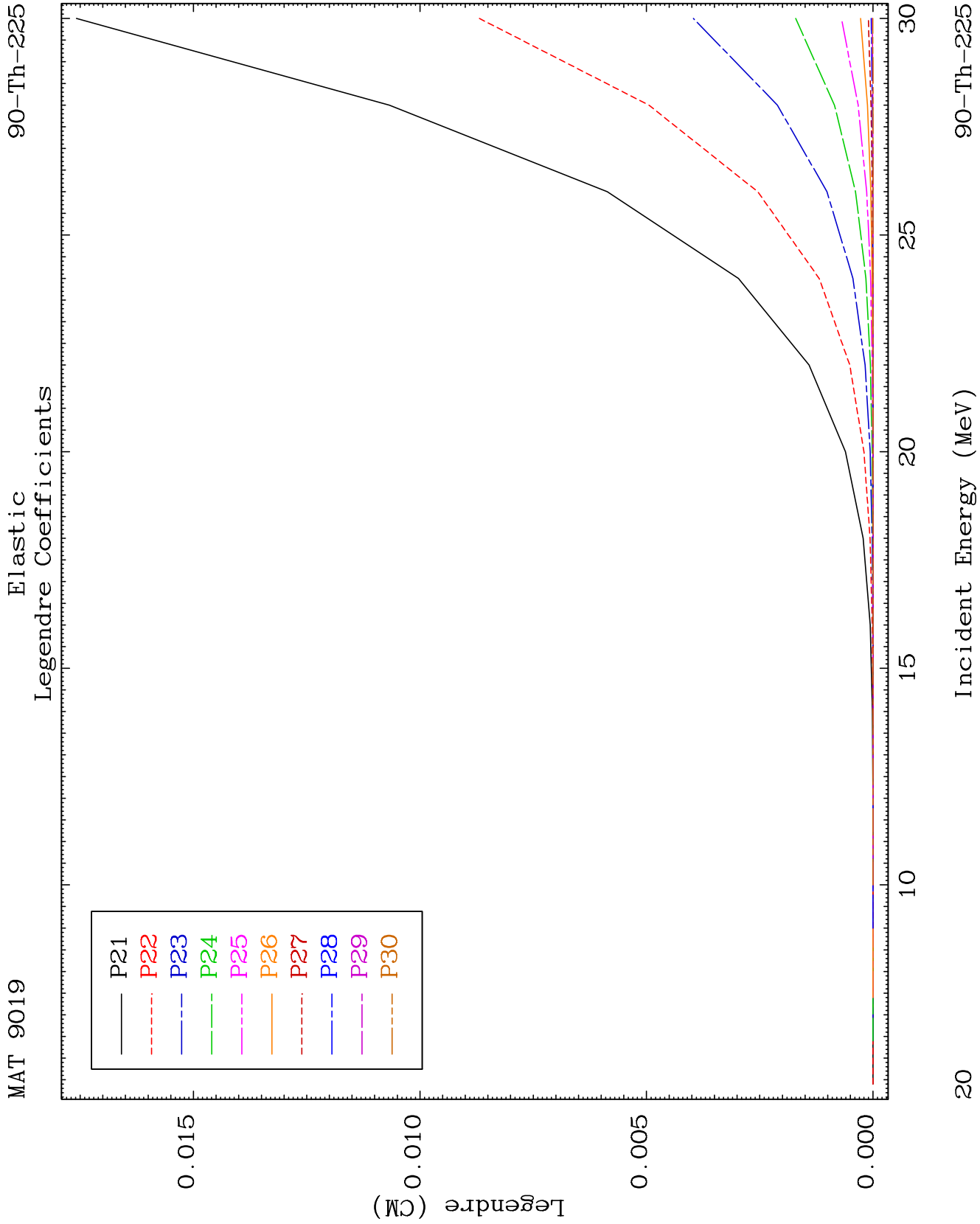
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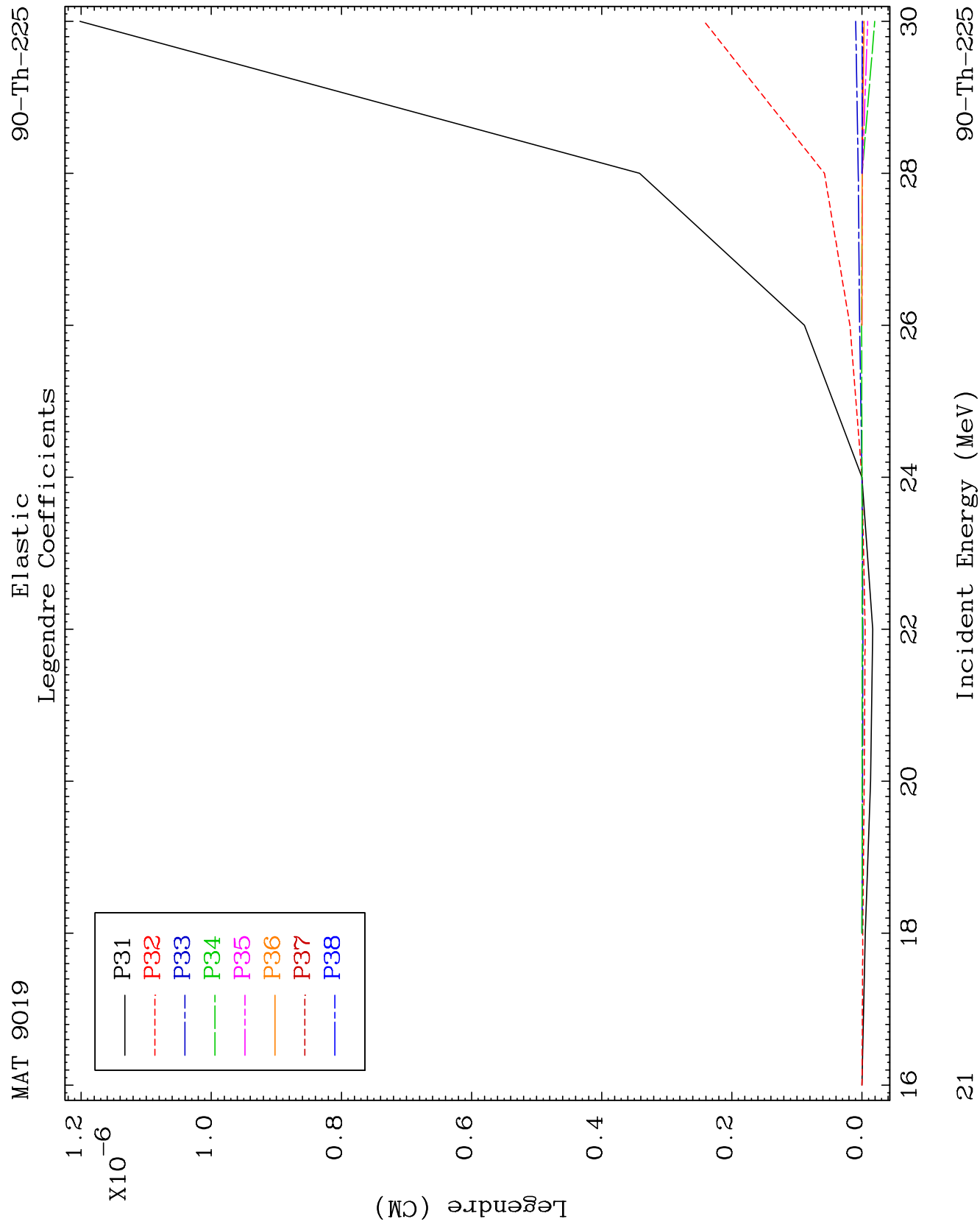








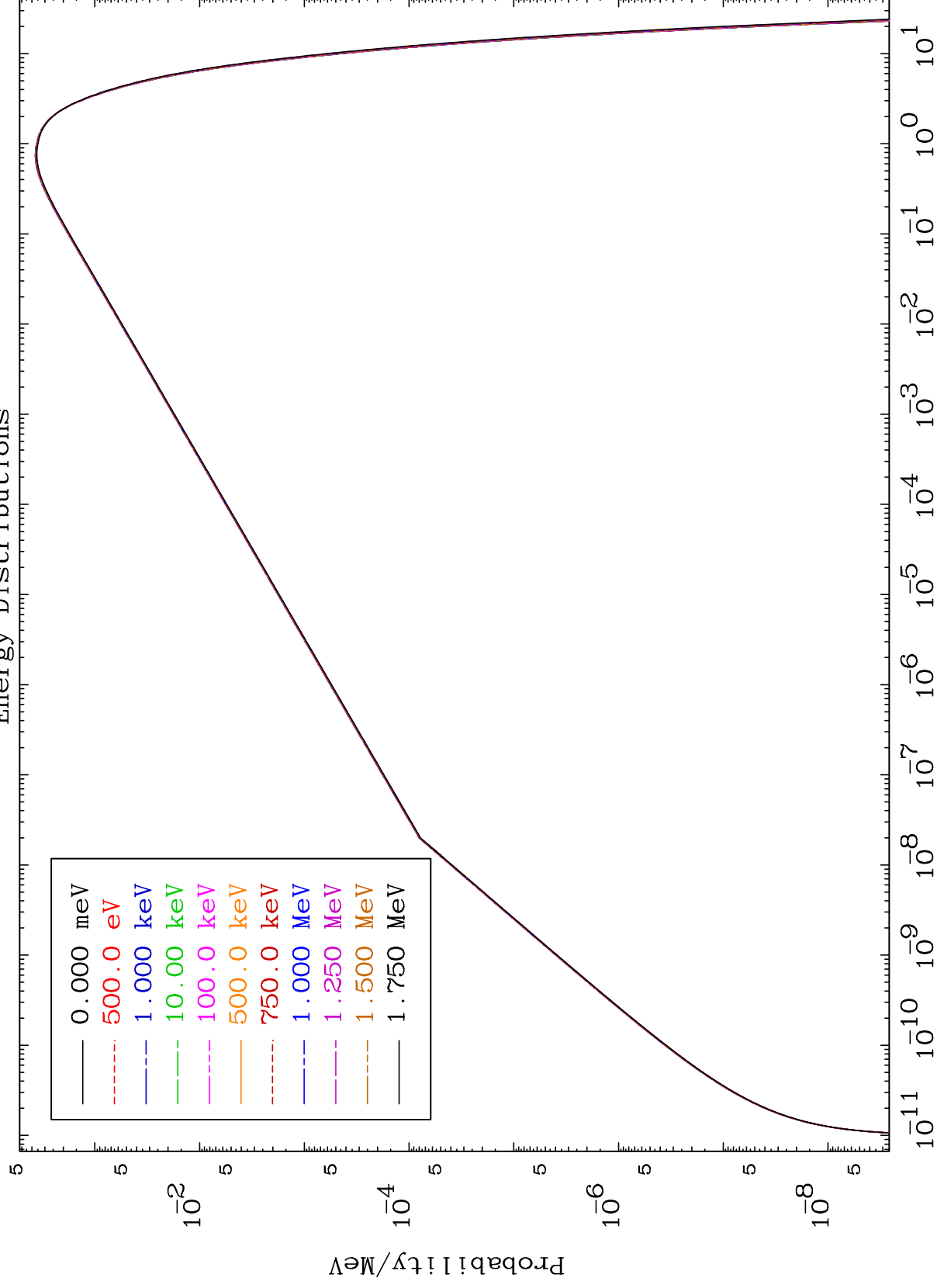




MAT 9019

Fission
Energy Distributions

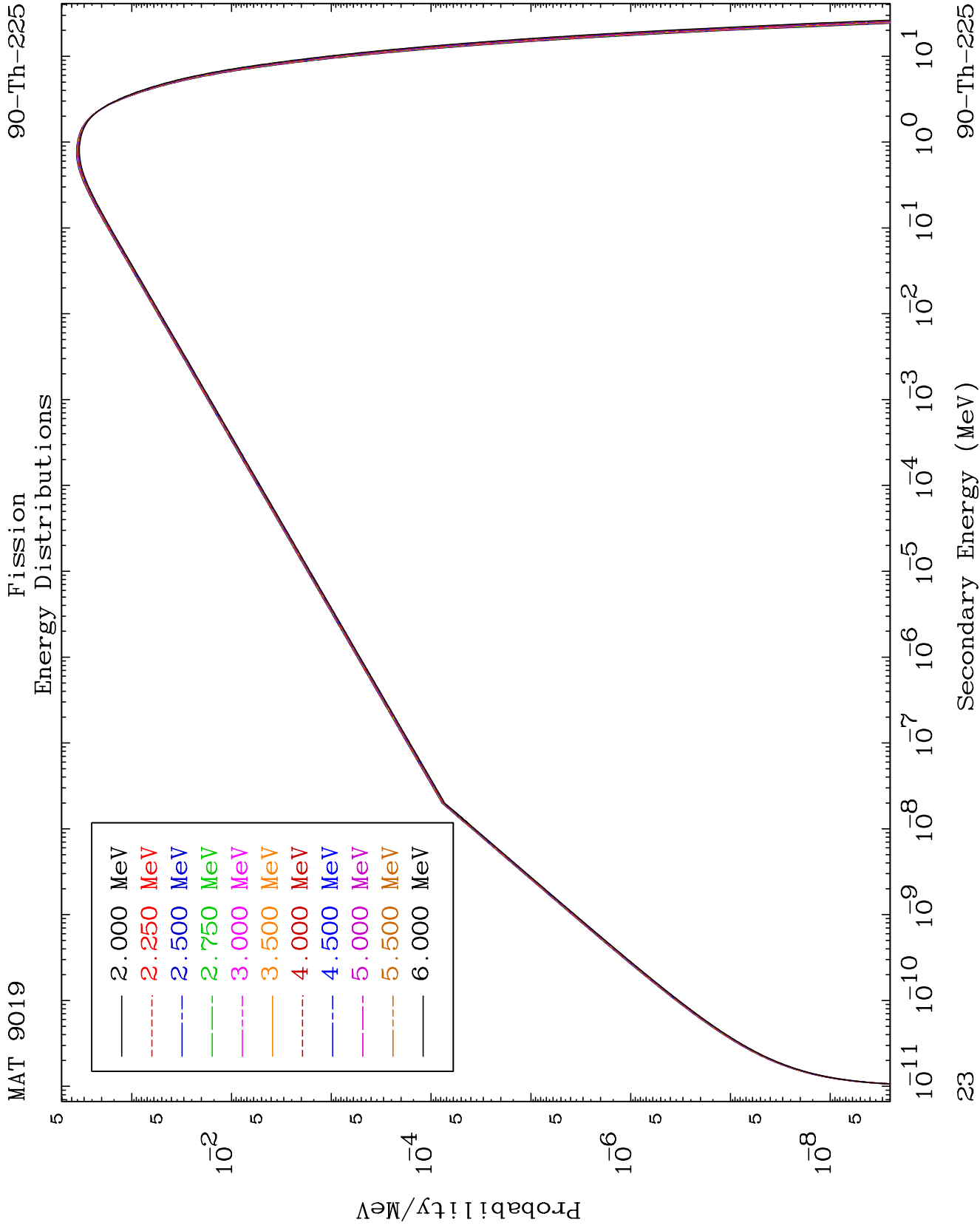
90-Th-225



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Secondary Energy (MeV)

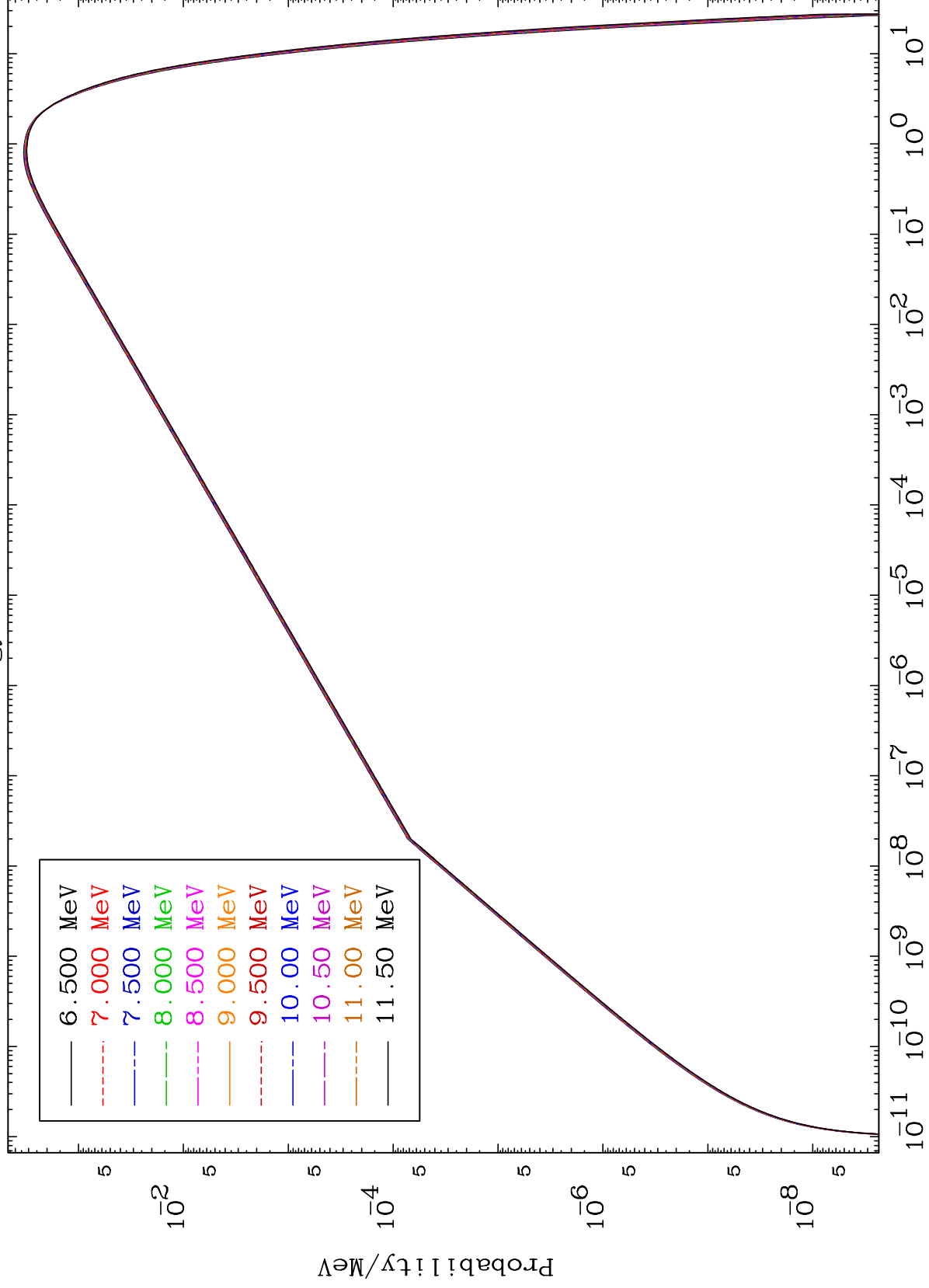
90-Th-225

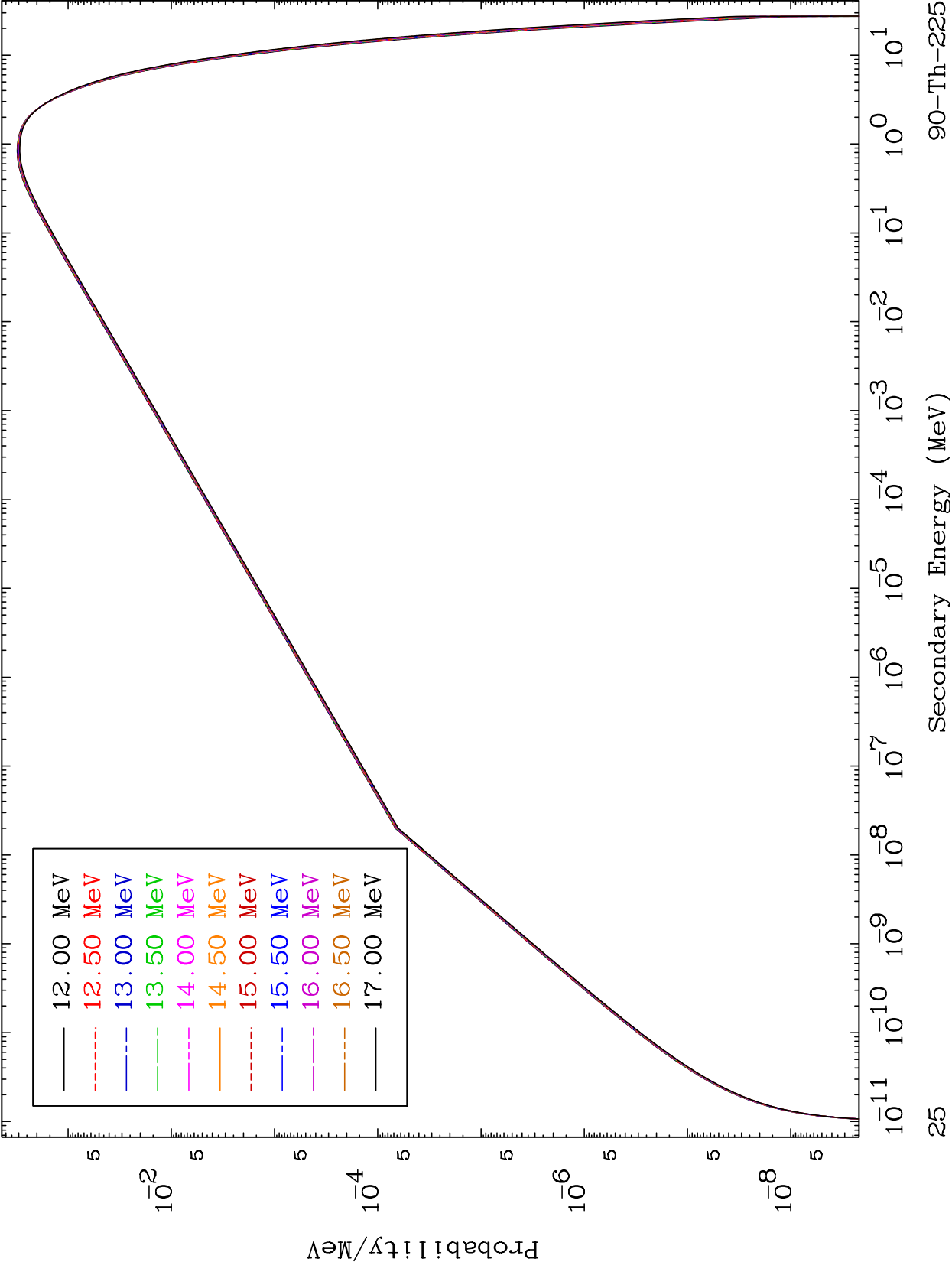


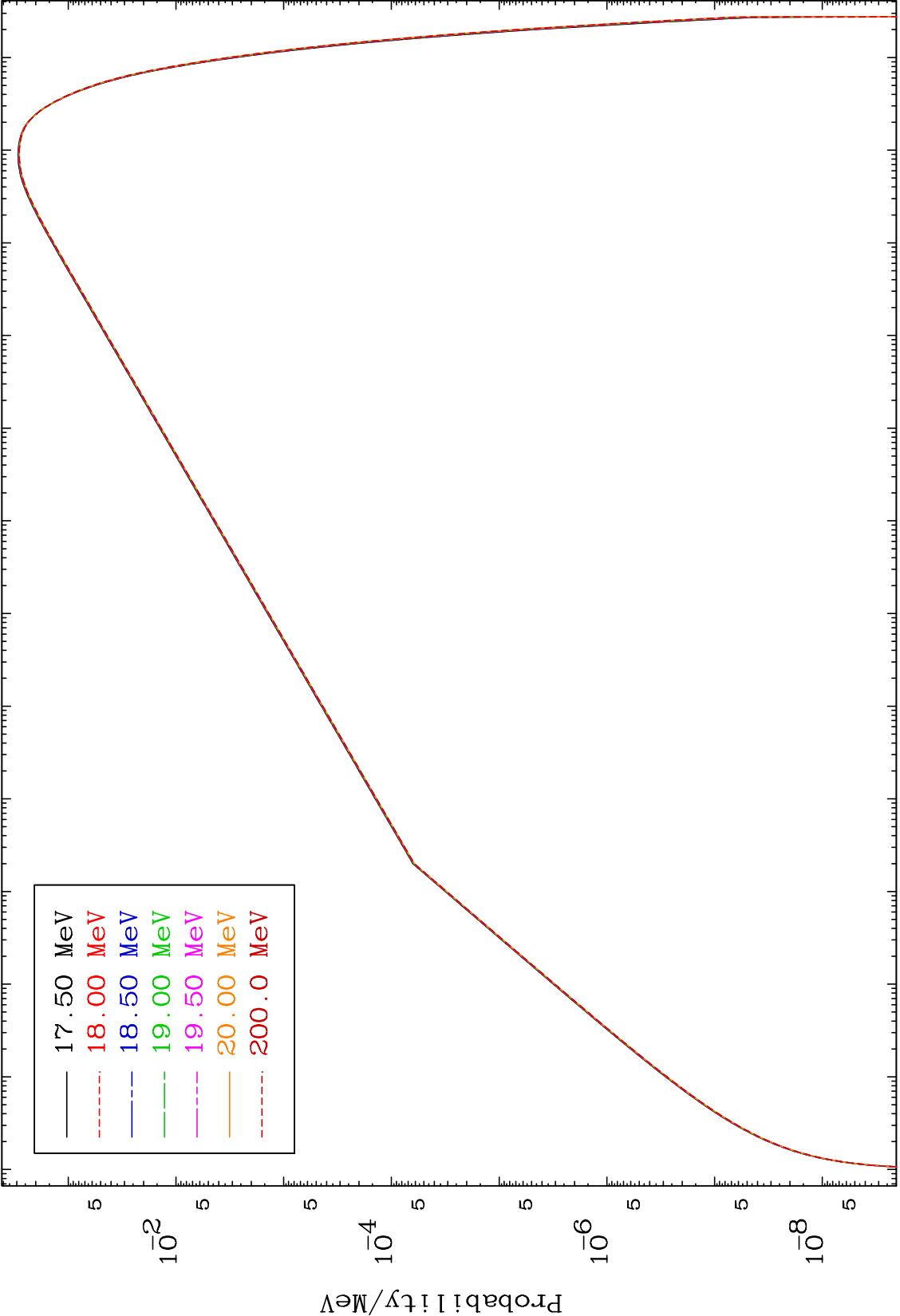
MAT 9019

Fission
Energy Distributions

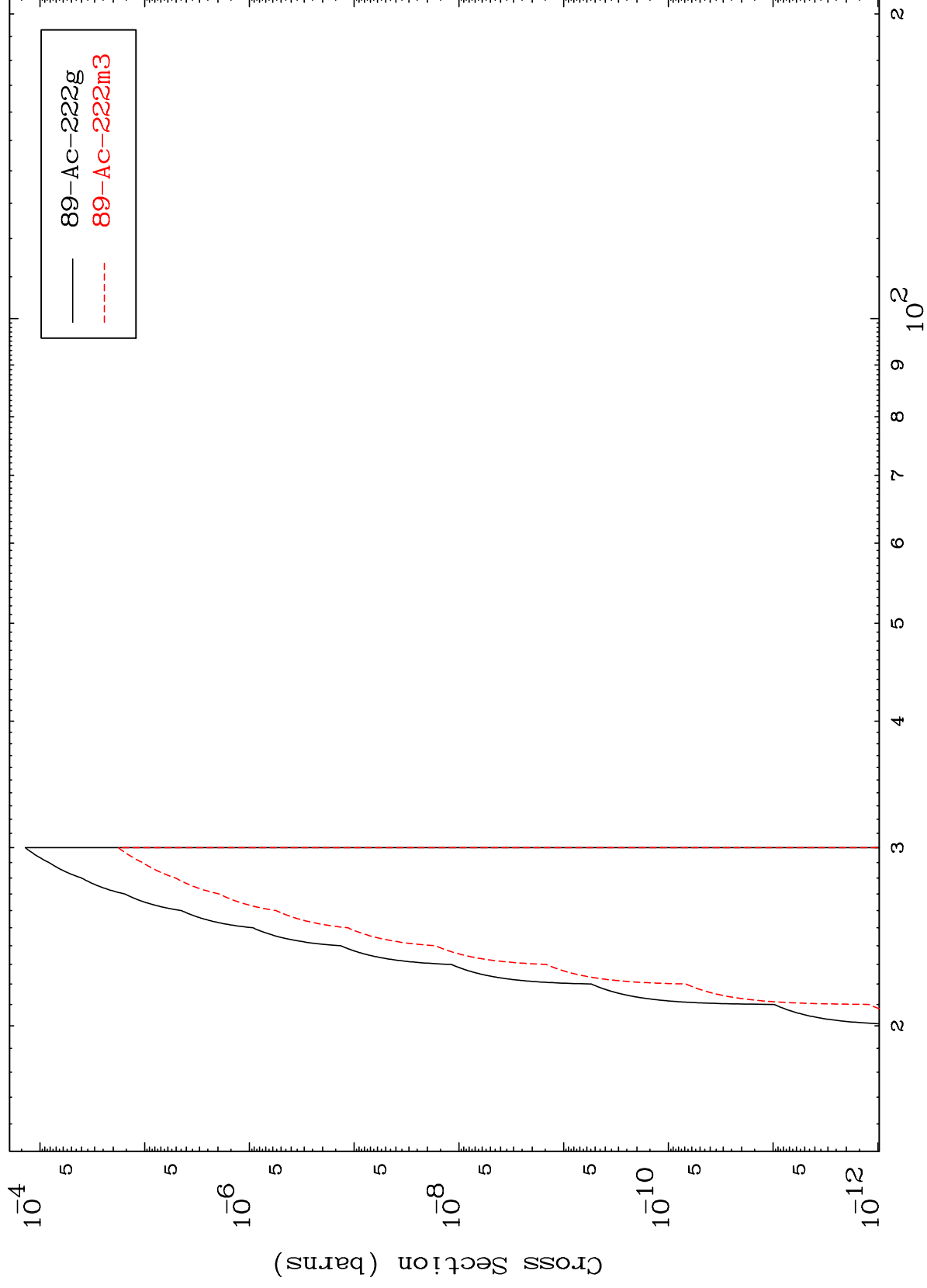
90-Th-225







(n,2n) d
Radionuclide Production Cross Section

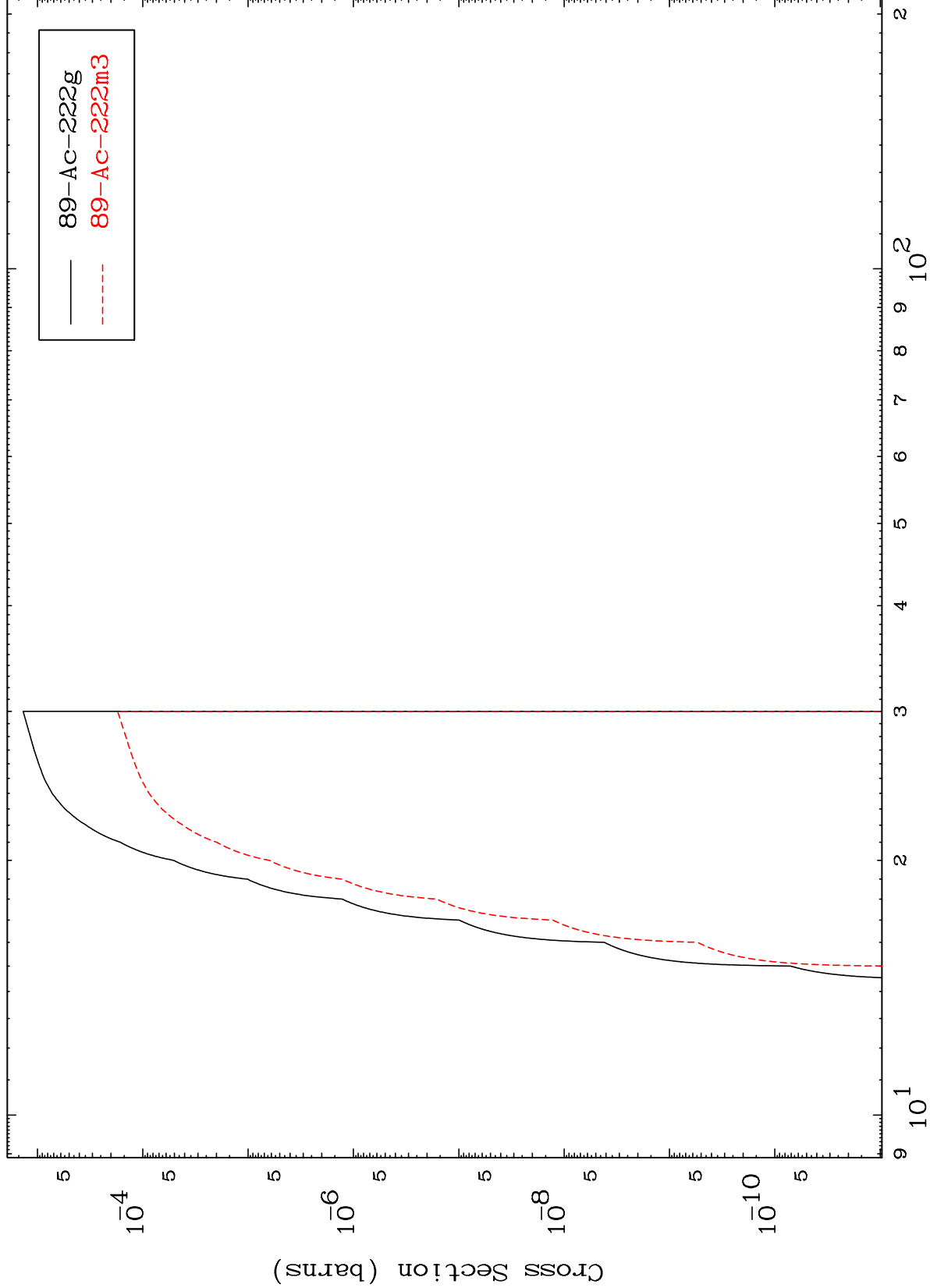


MAT 9019

90-Th-225

(n,n') t

Radionuclide Production Cross Section



90-Th-225

Incident Energy (MeV)

28

(n,3n) p
Radionuclide Production Cross Section

