

Program EVALPLOT
(Version 2021-1)

by

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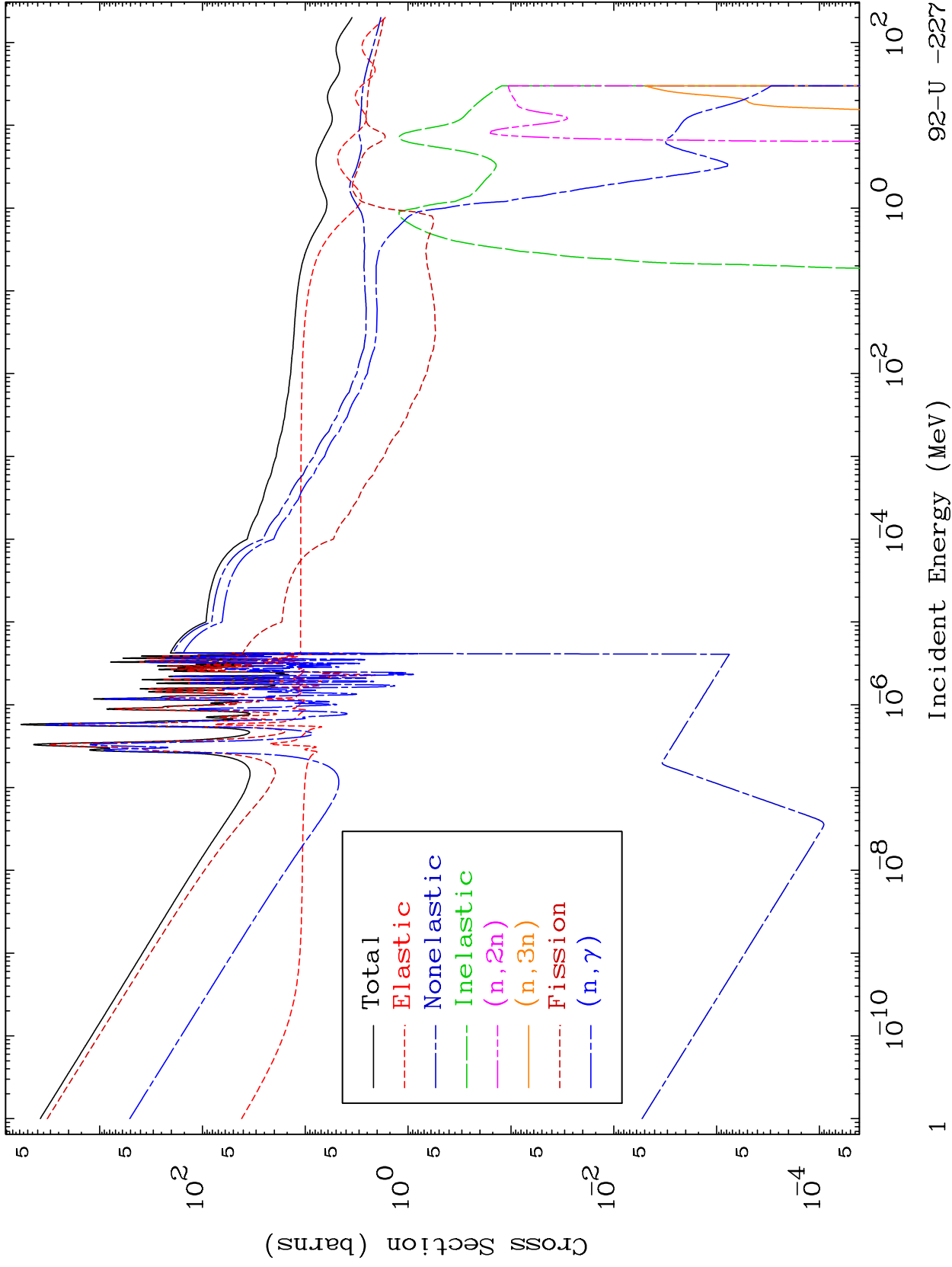
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9204

Neutron Major
293 Kelvin Cross Sections

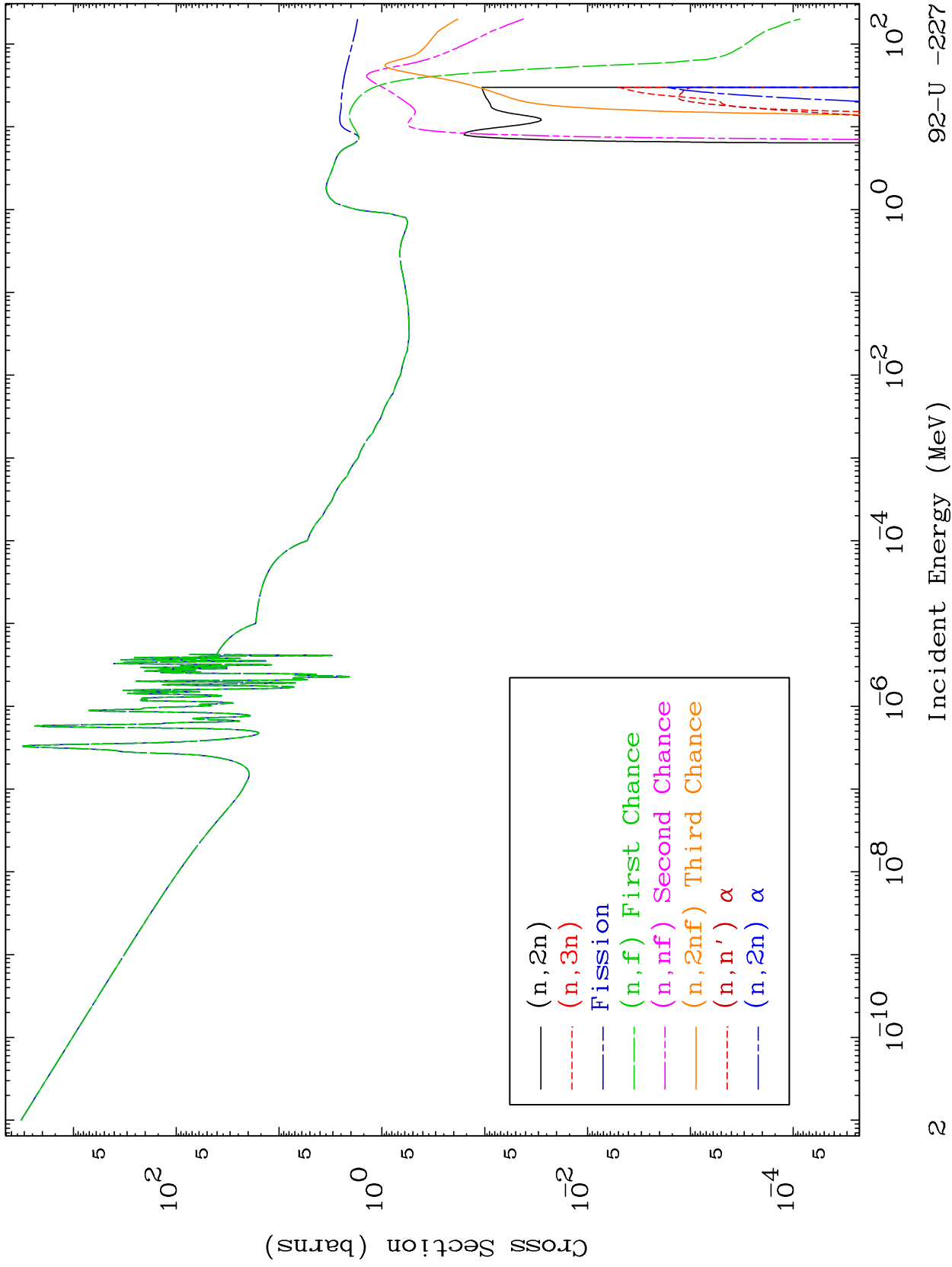
92-U -227



MAT 9204

Neutron Absorption
293 Kelvin Cross Sections

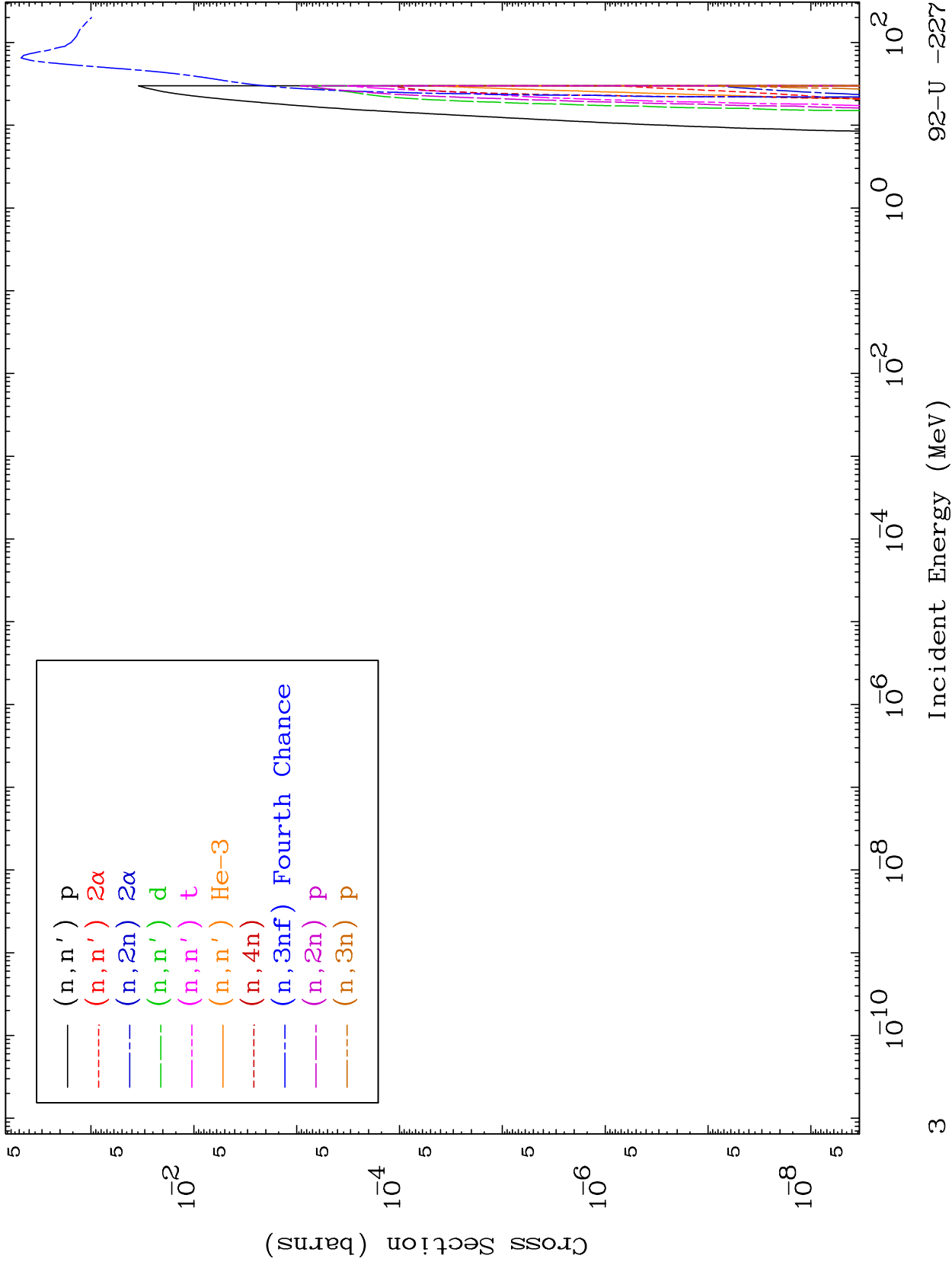
92-U -227



MAT 9204

Neutron Absorption
293 Kelvin Cross Sections

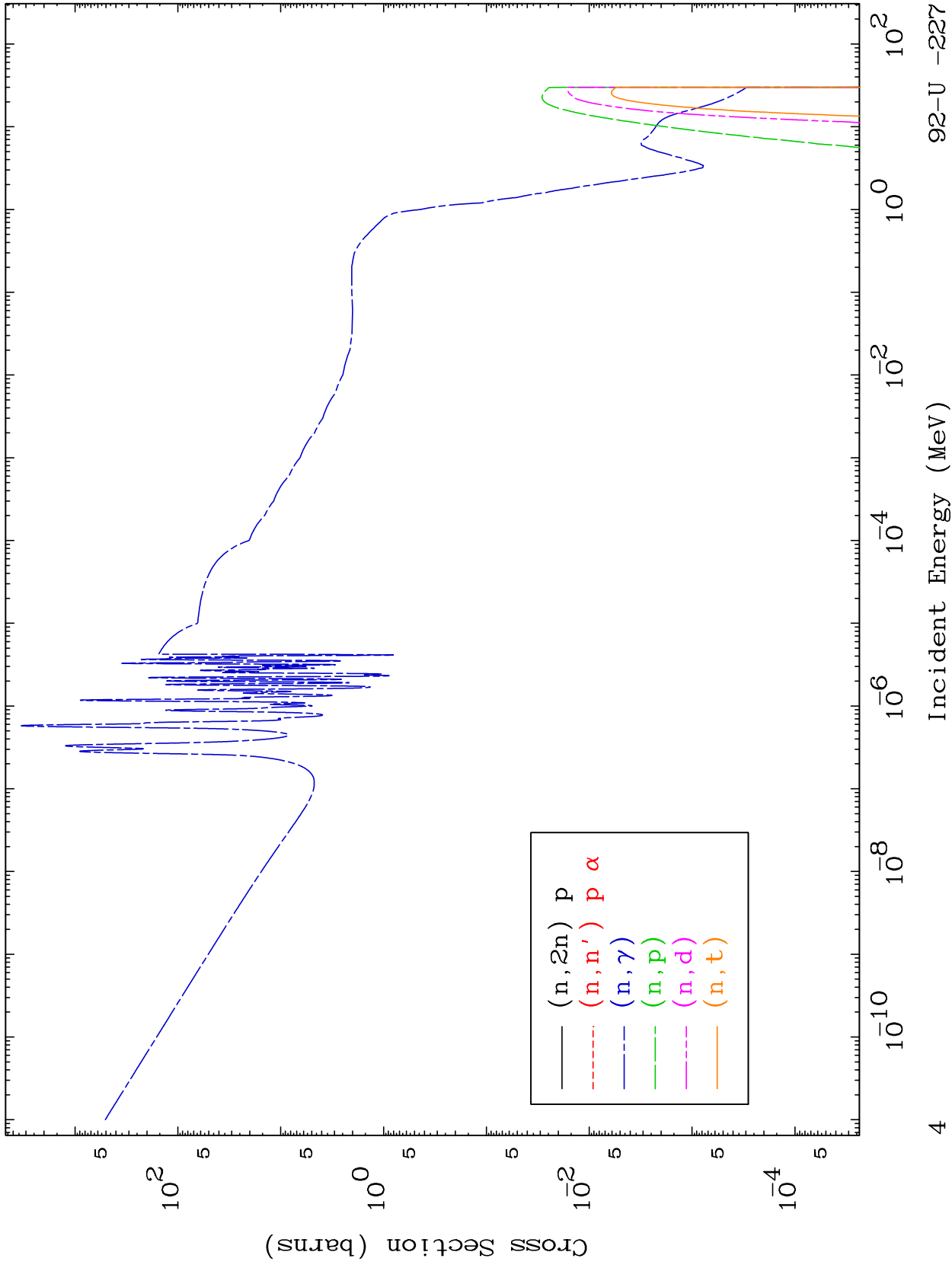
92-U -227

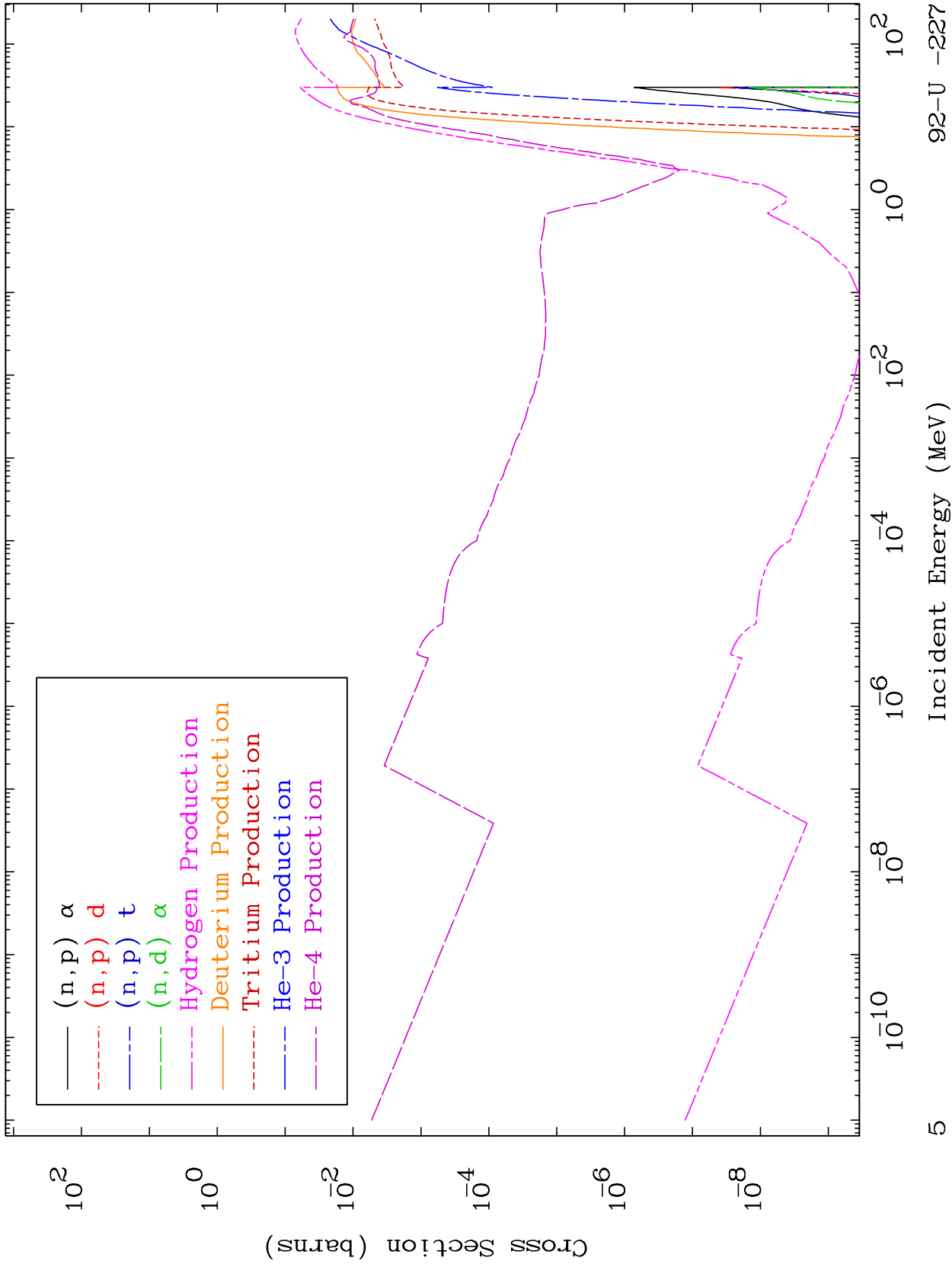


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Neutron Absorption
293 Kelvin Cross Sections

92-U -227

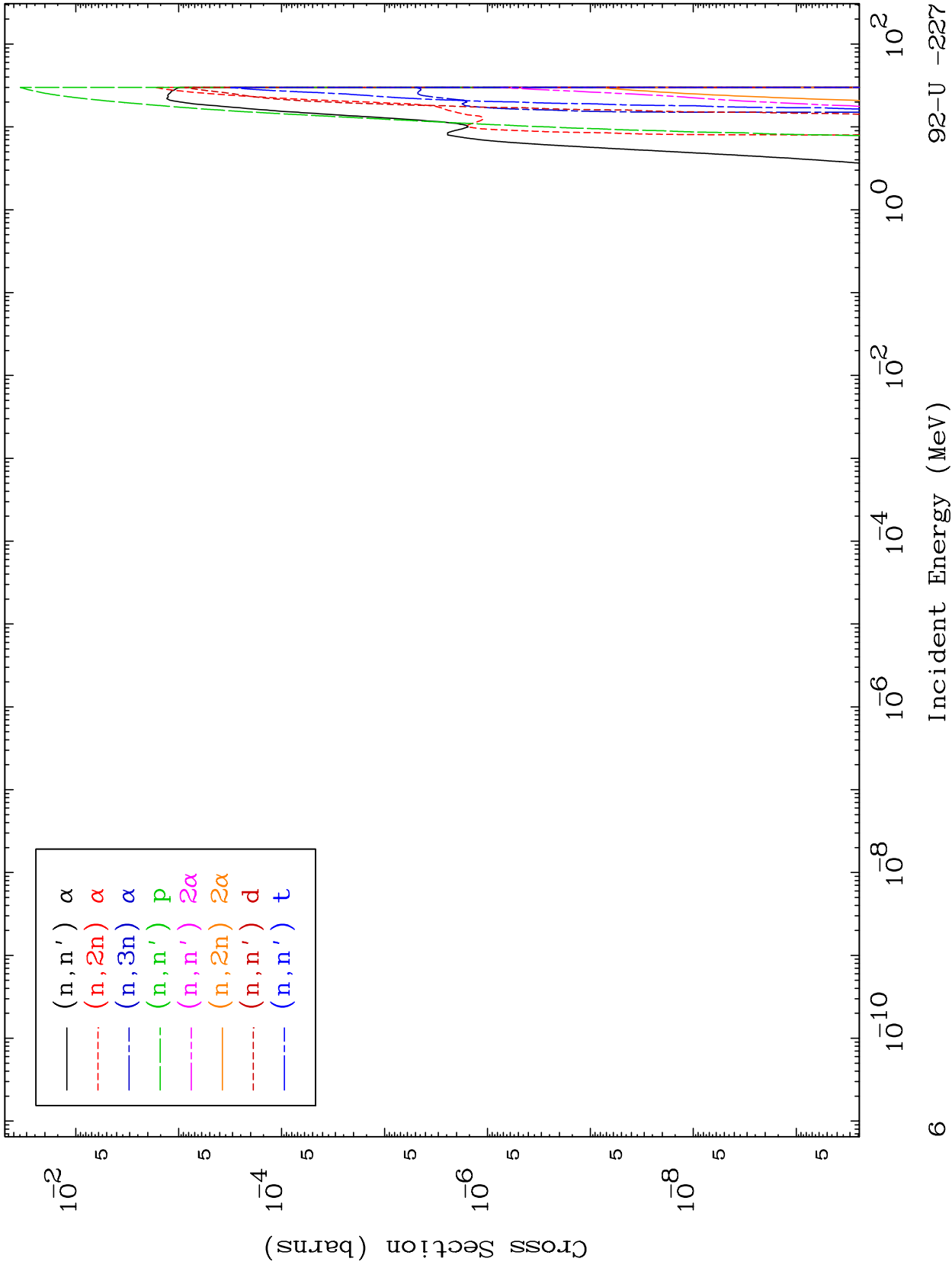




MAT 9204

293 Charged Particle
Kelvin Cross Sections

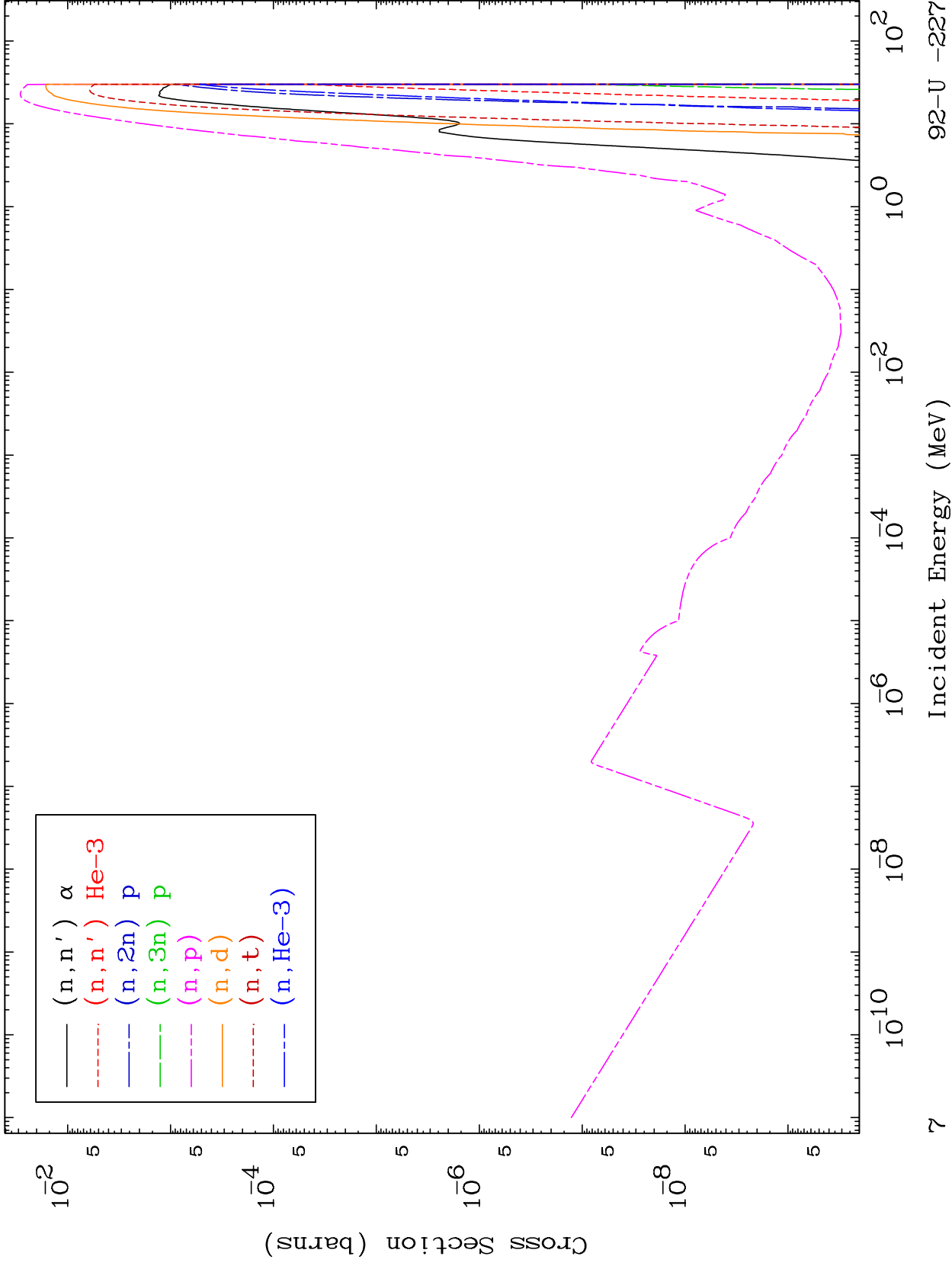
92-U -227

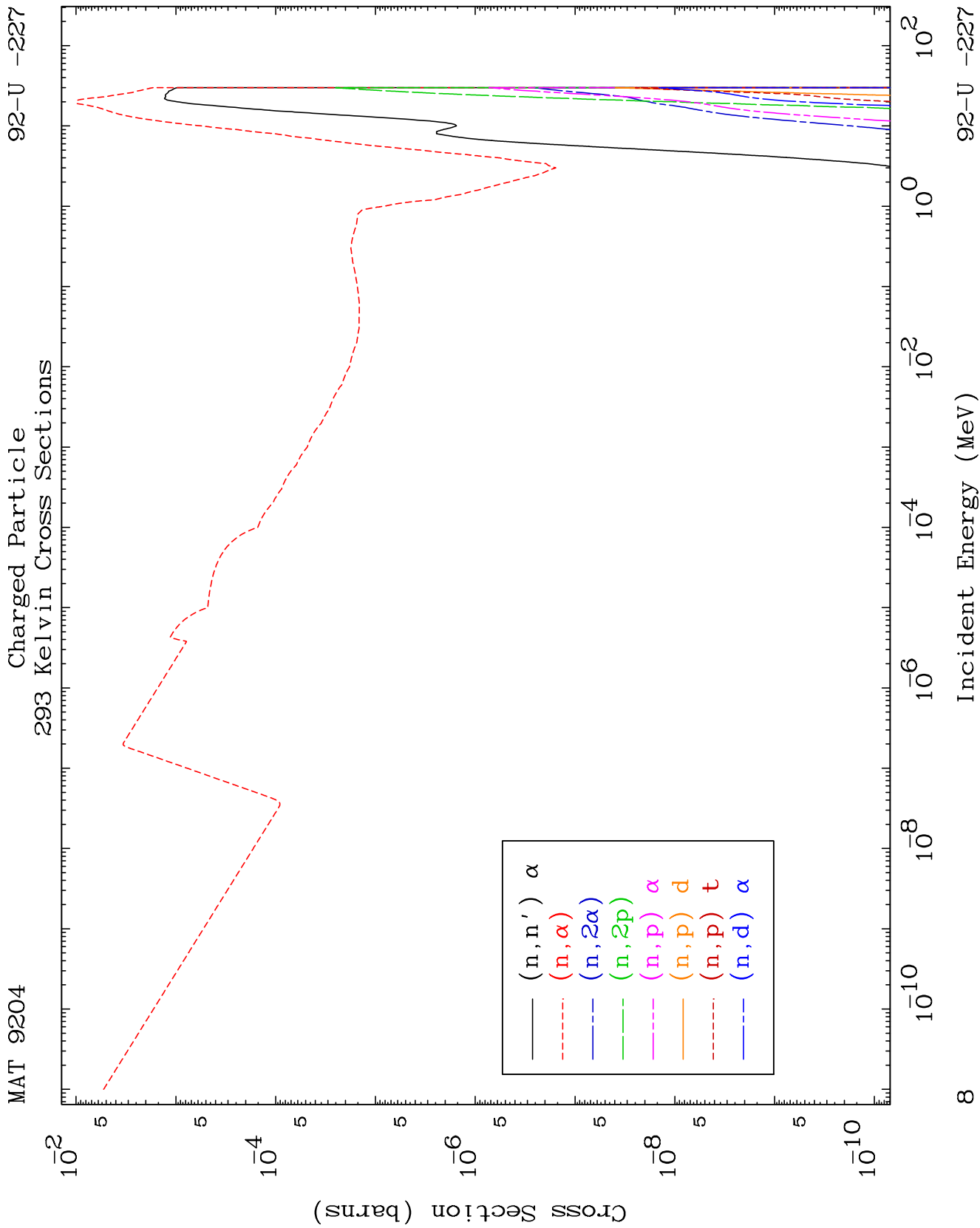


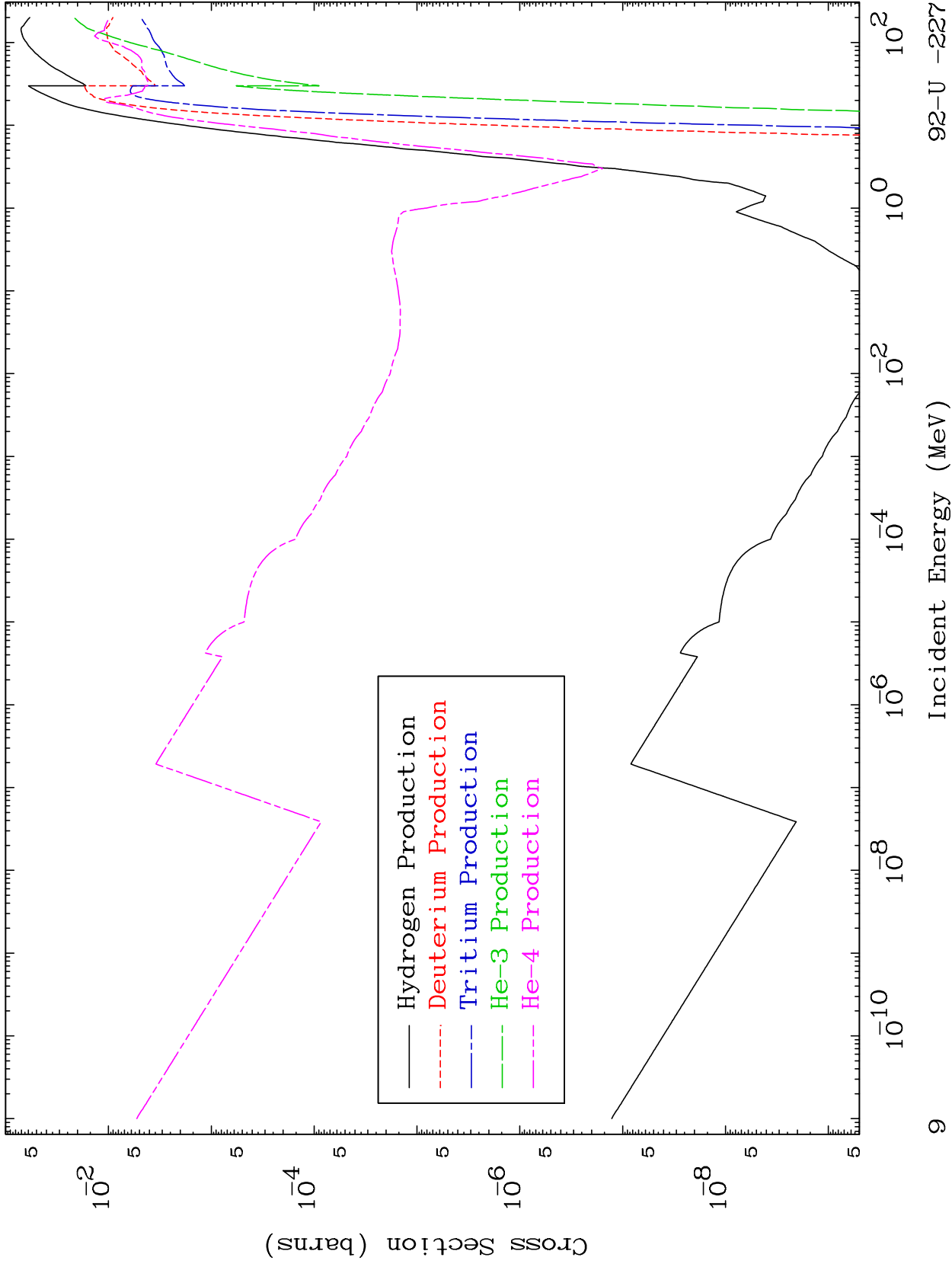
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Charged Particle
293 Kelvin Cross Sections

92-U -227



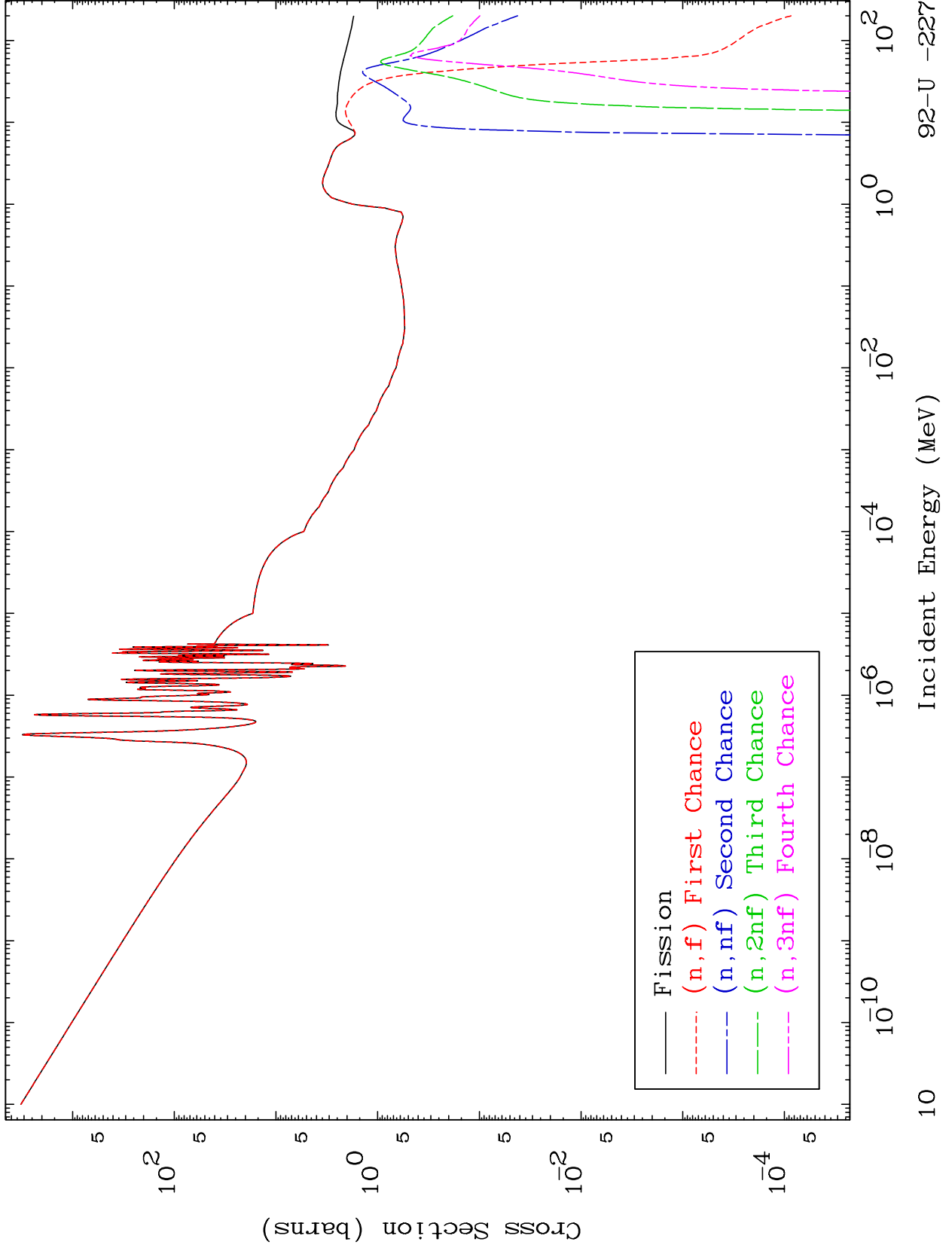


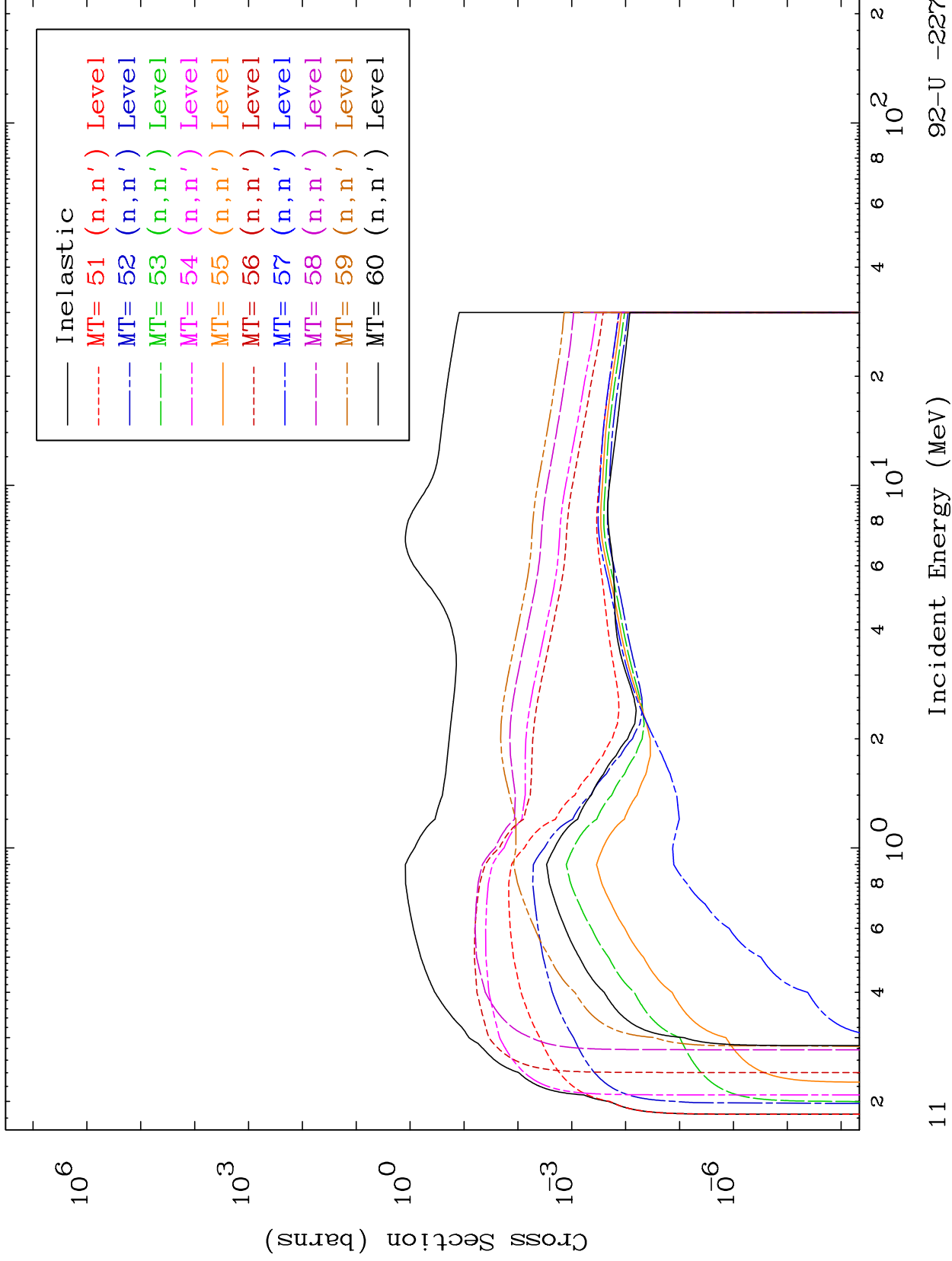


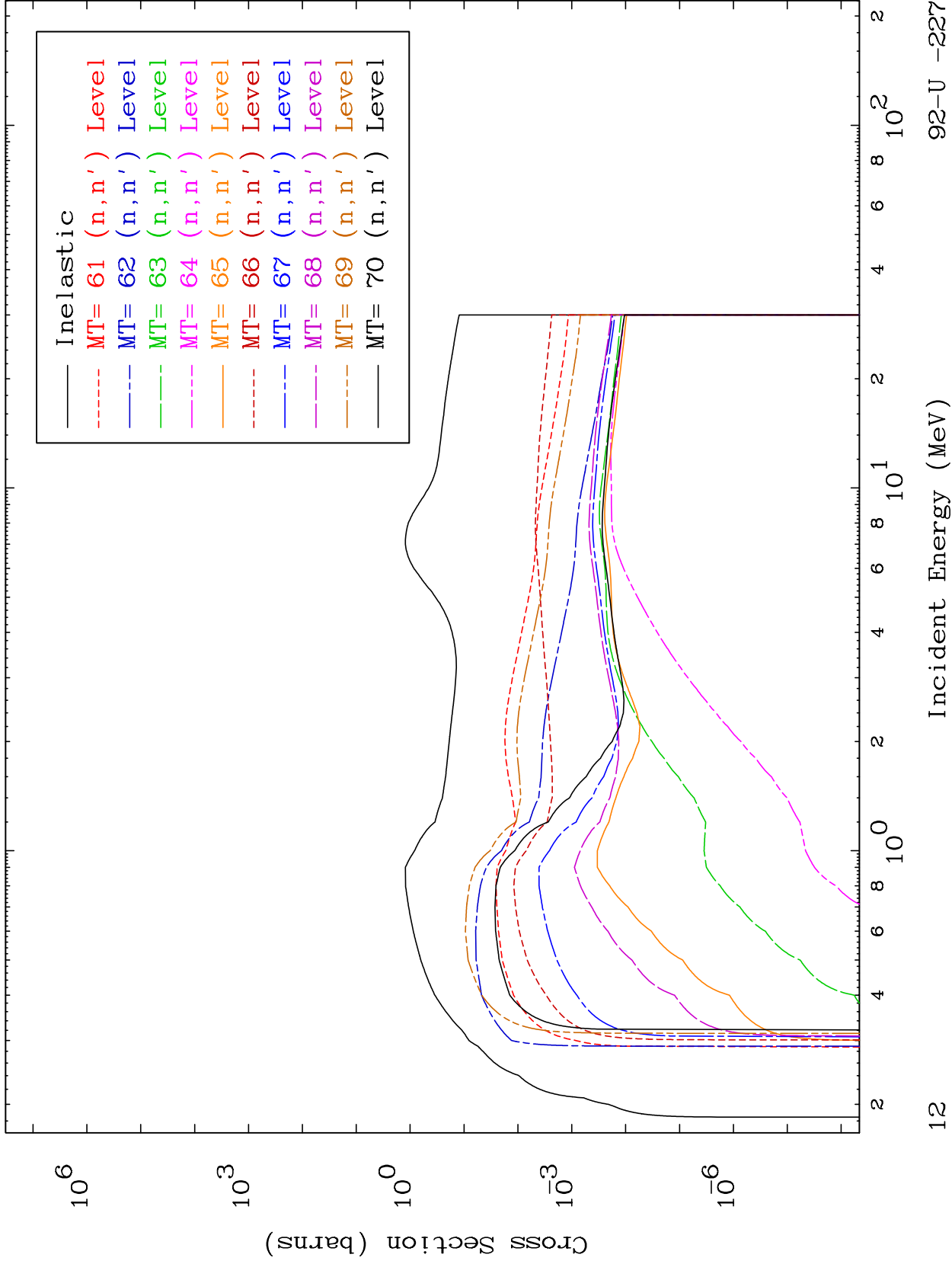
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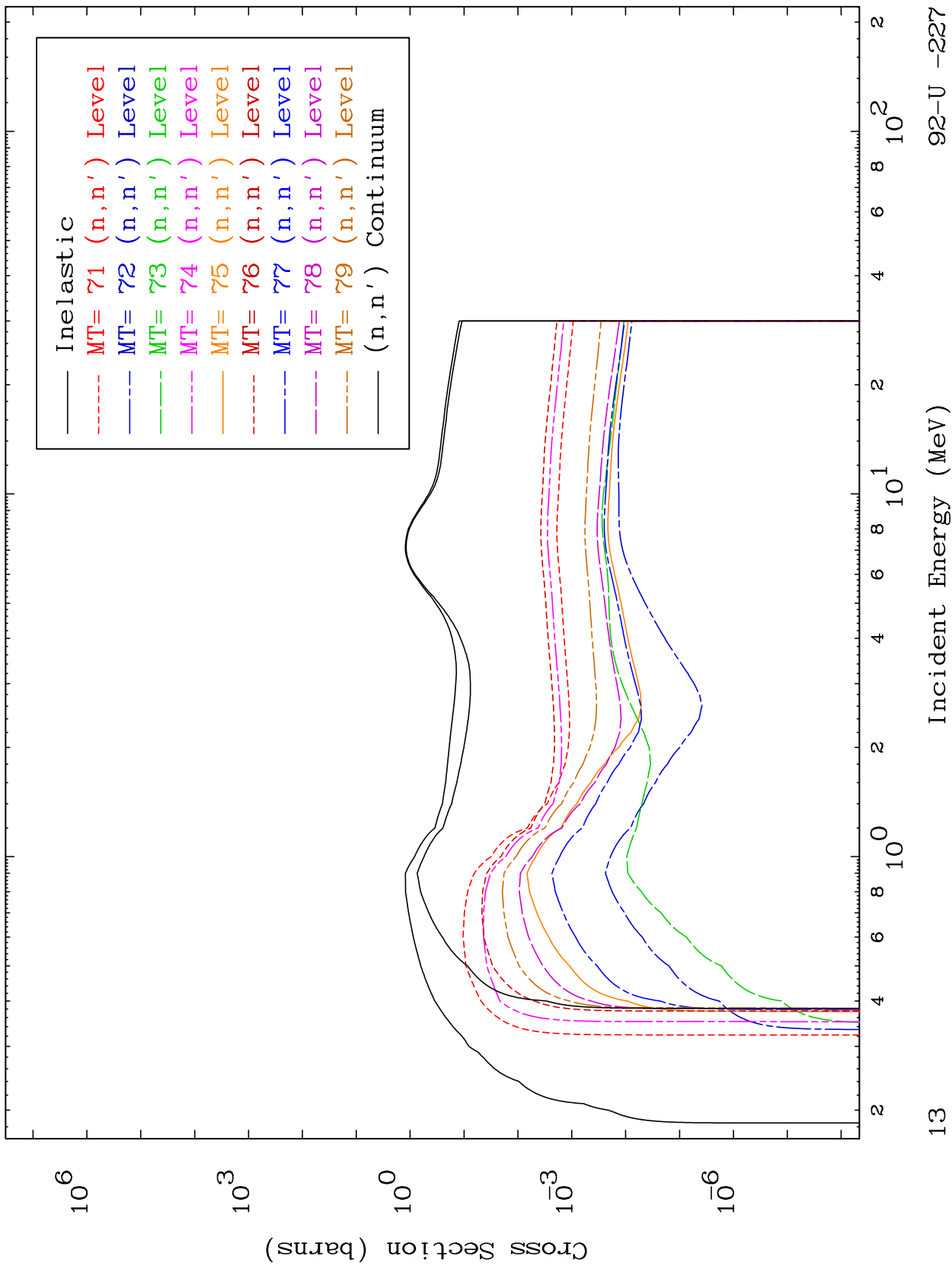
Fission
293 Kelvin Cross Sections

92-U -227





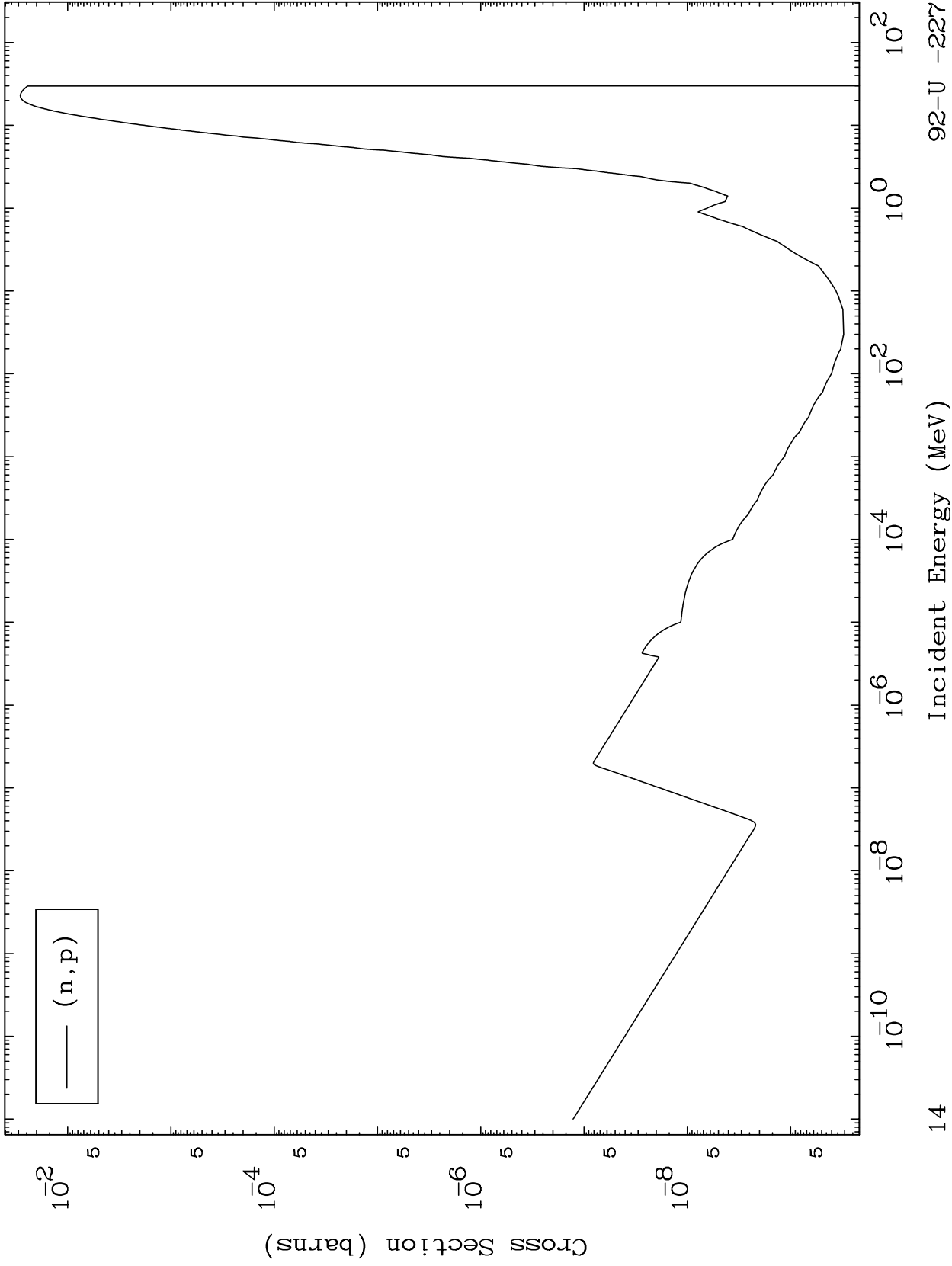




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(n,p) Levels
293 Kelvin Cross Sections

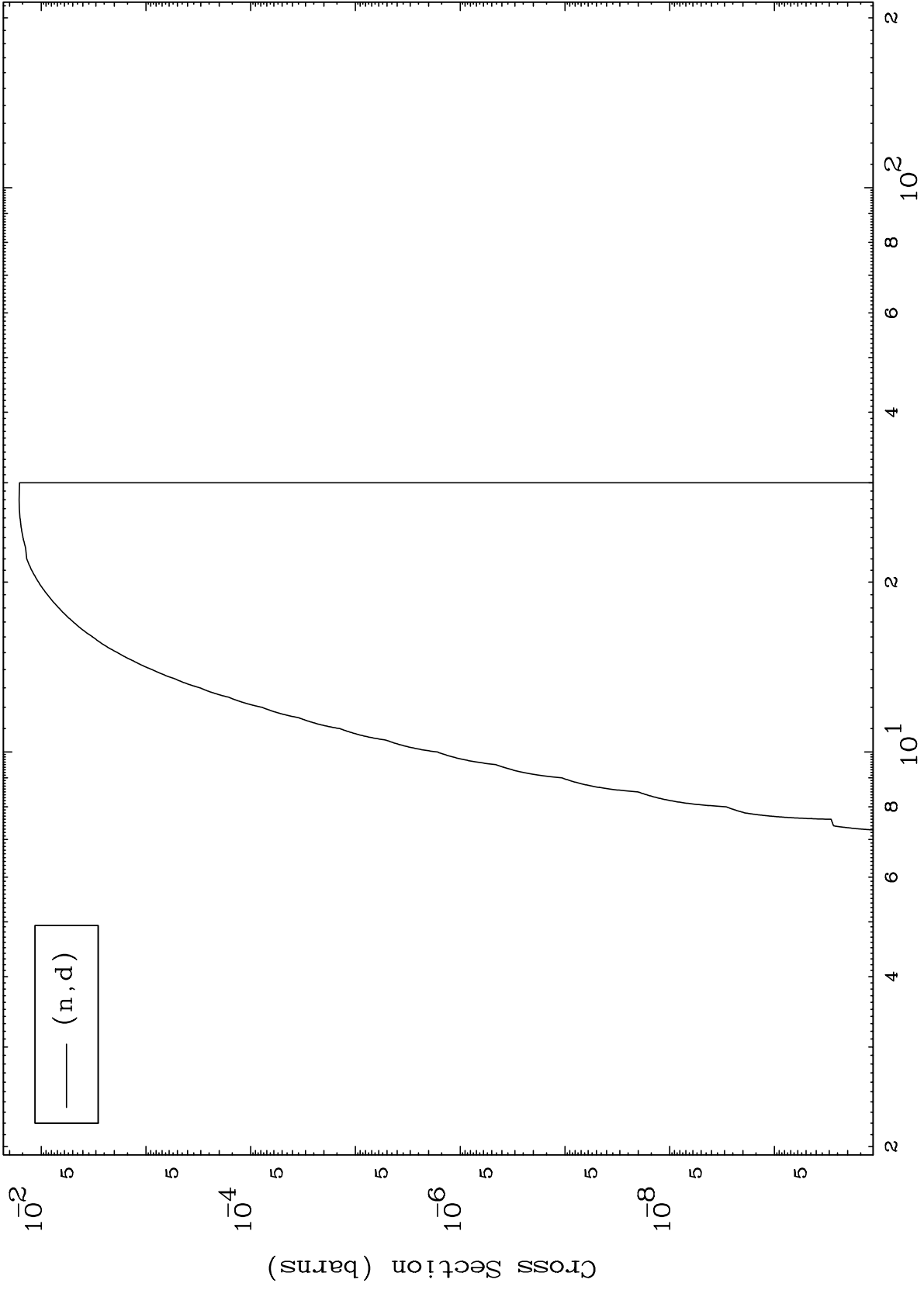
92-U -227



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(n,d) Levels
293 Kelvin Cross Sections

92-U -227



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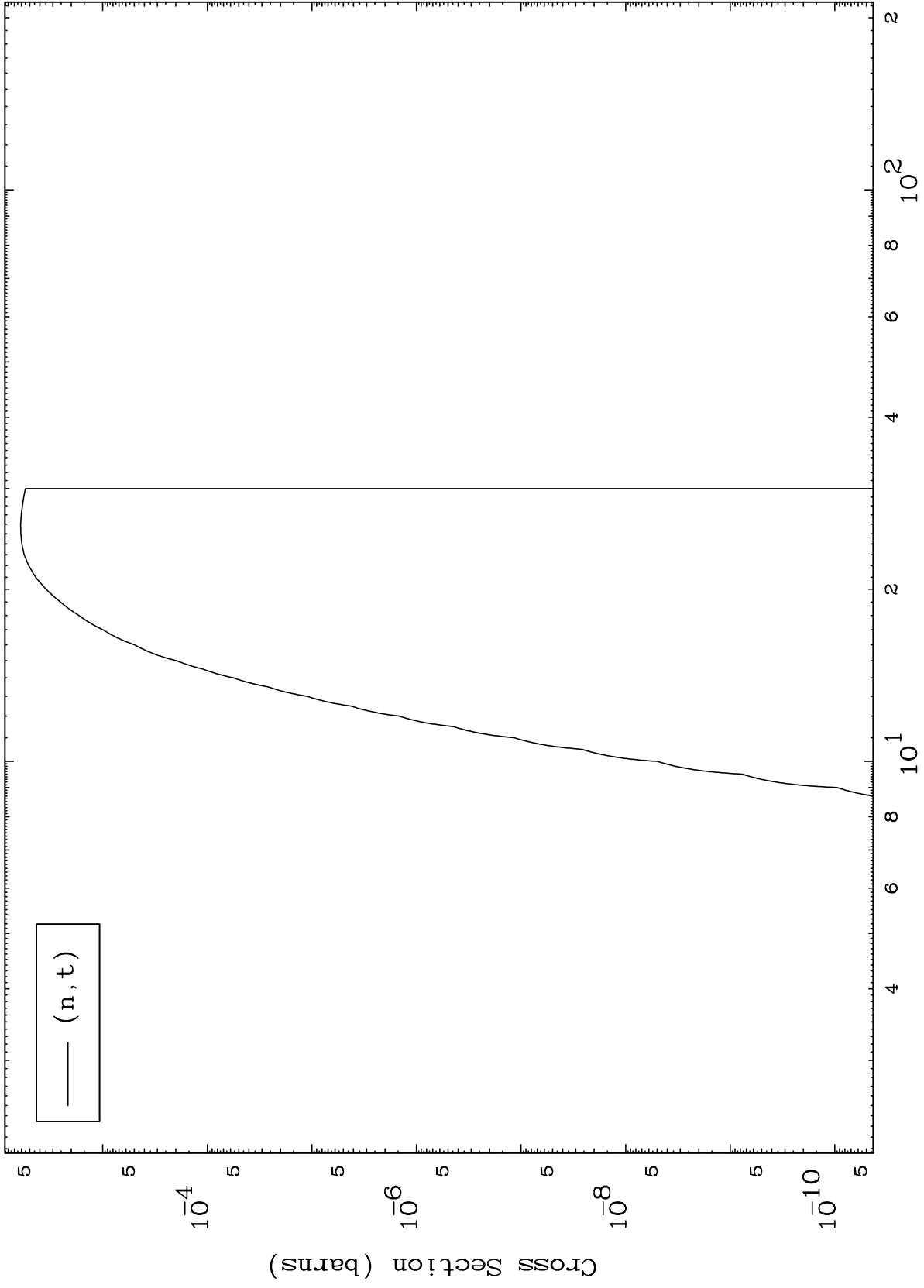
Incident Energy (MeV)

92-U -227

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(n,t) Levels
293 Kelvin Cross Sections

92-U -227



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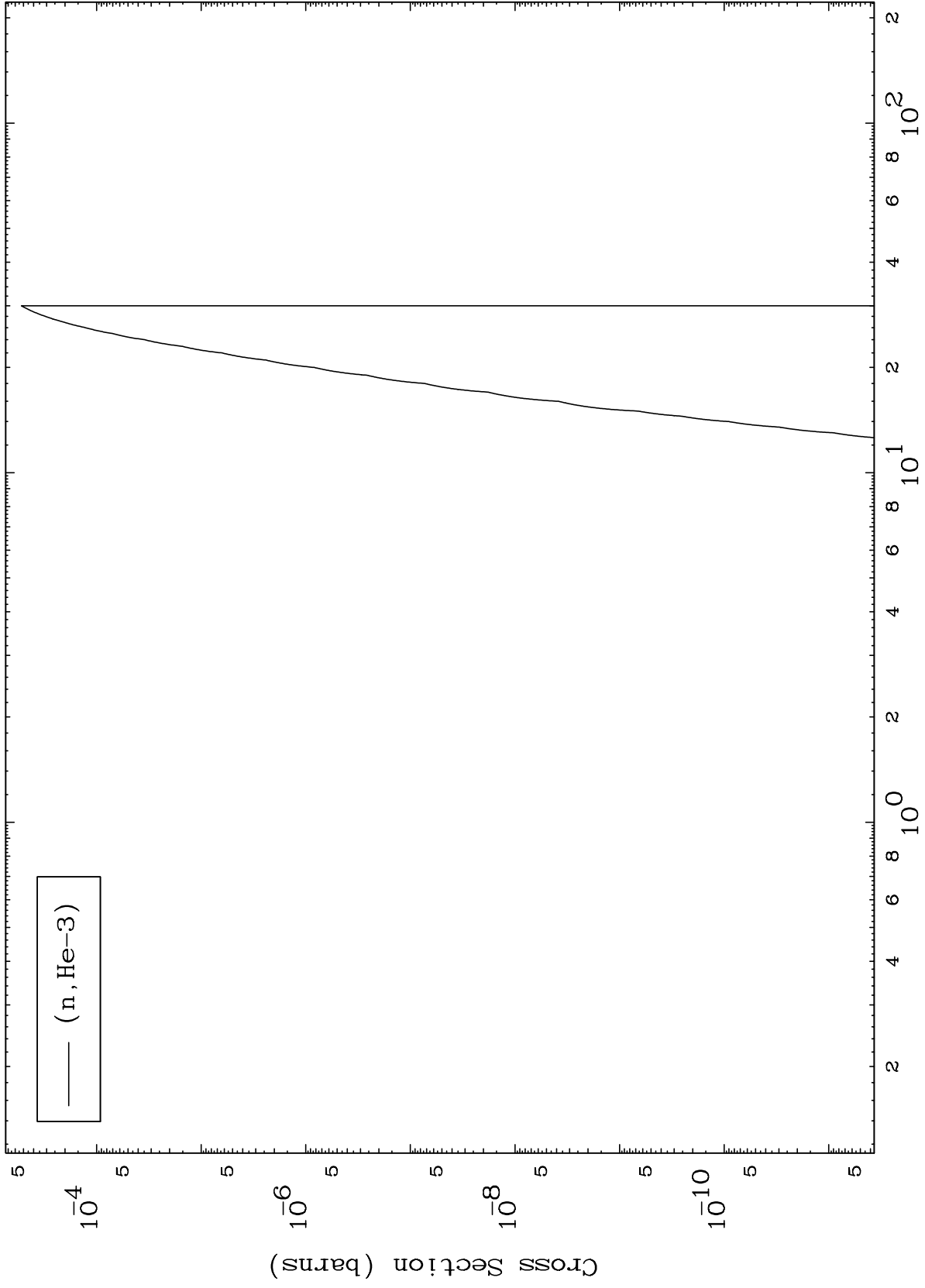
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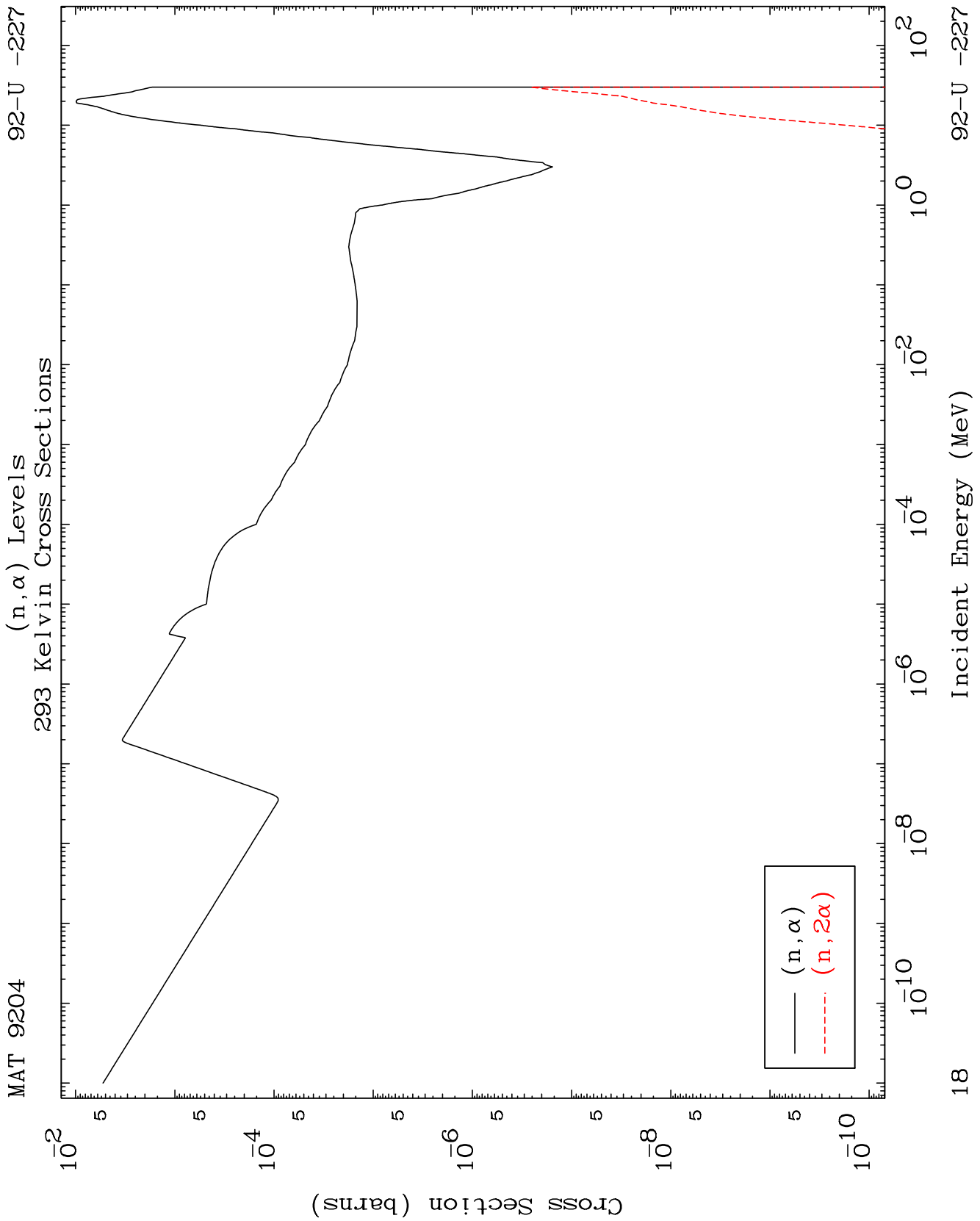
92-U -227

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(n,He3) Levels
293 Kelvin Cross Sections

92-U -227

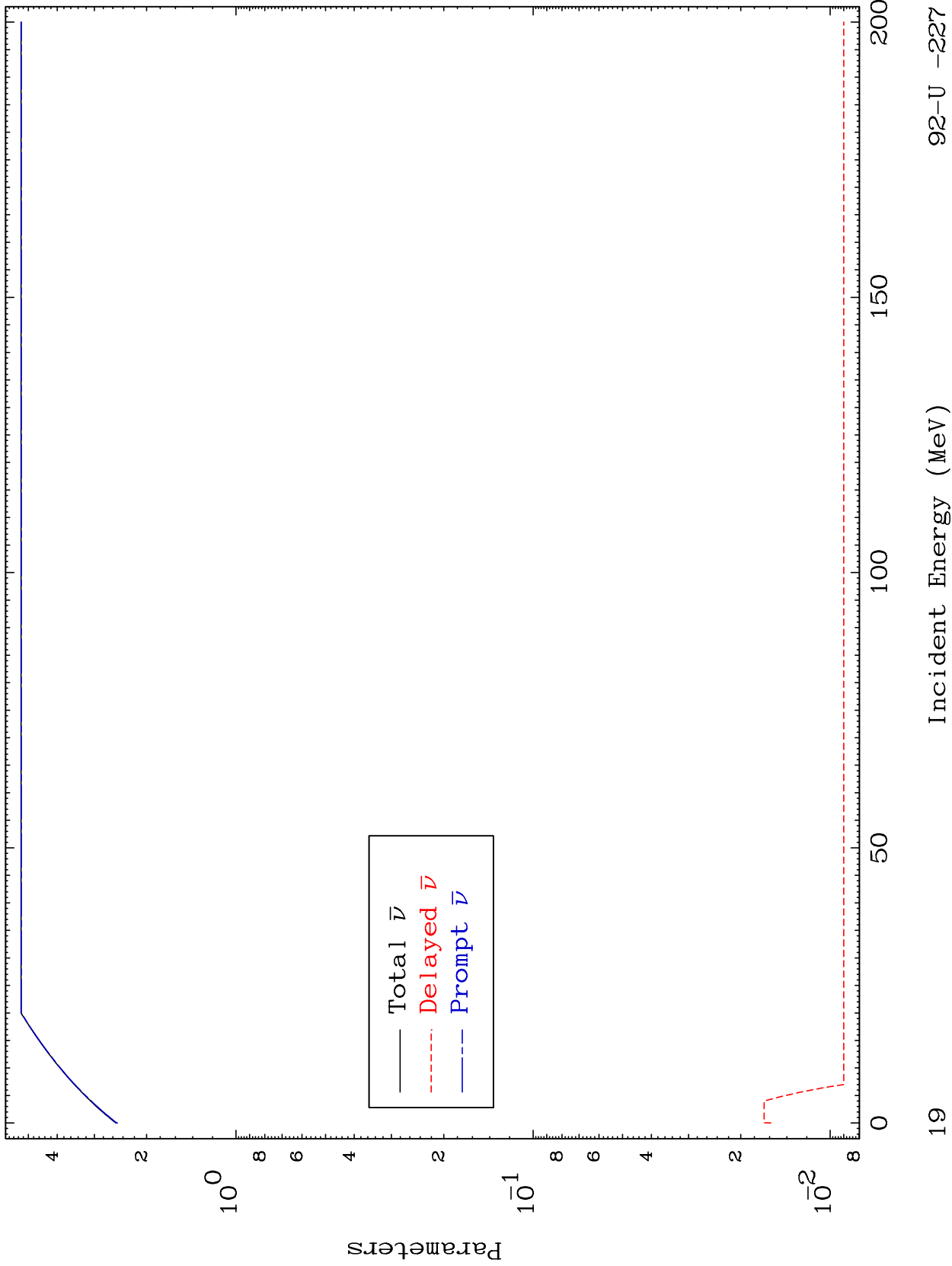




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Energy Release
Parameters

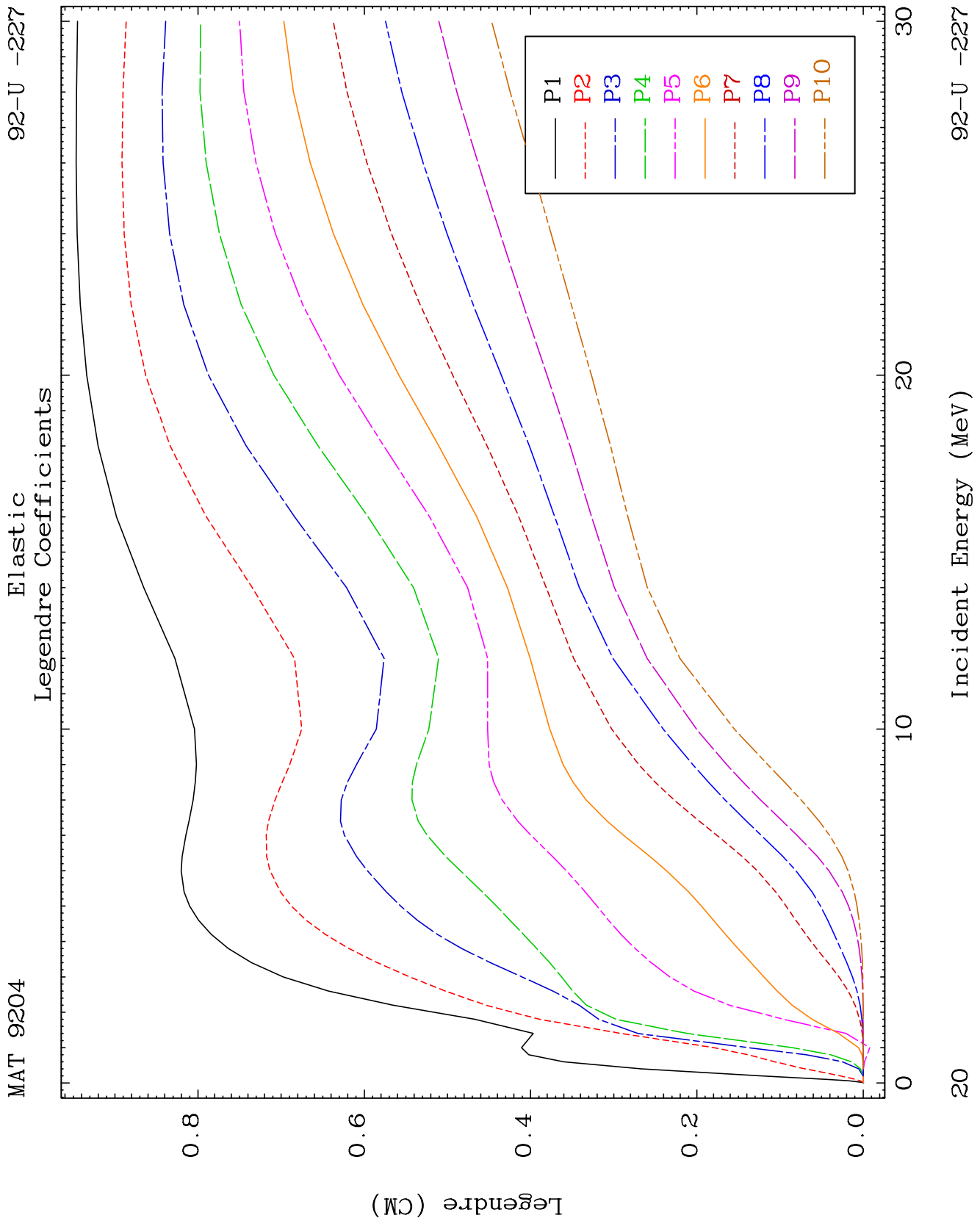
92-U -227



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Incident Energy (MeV)

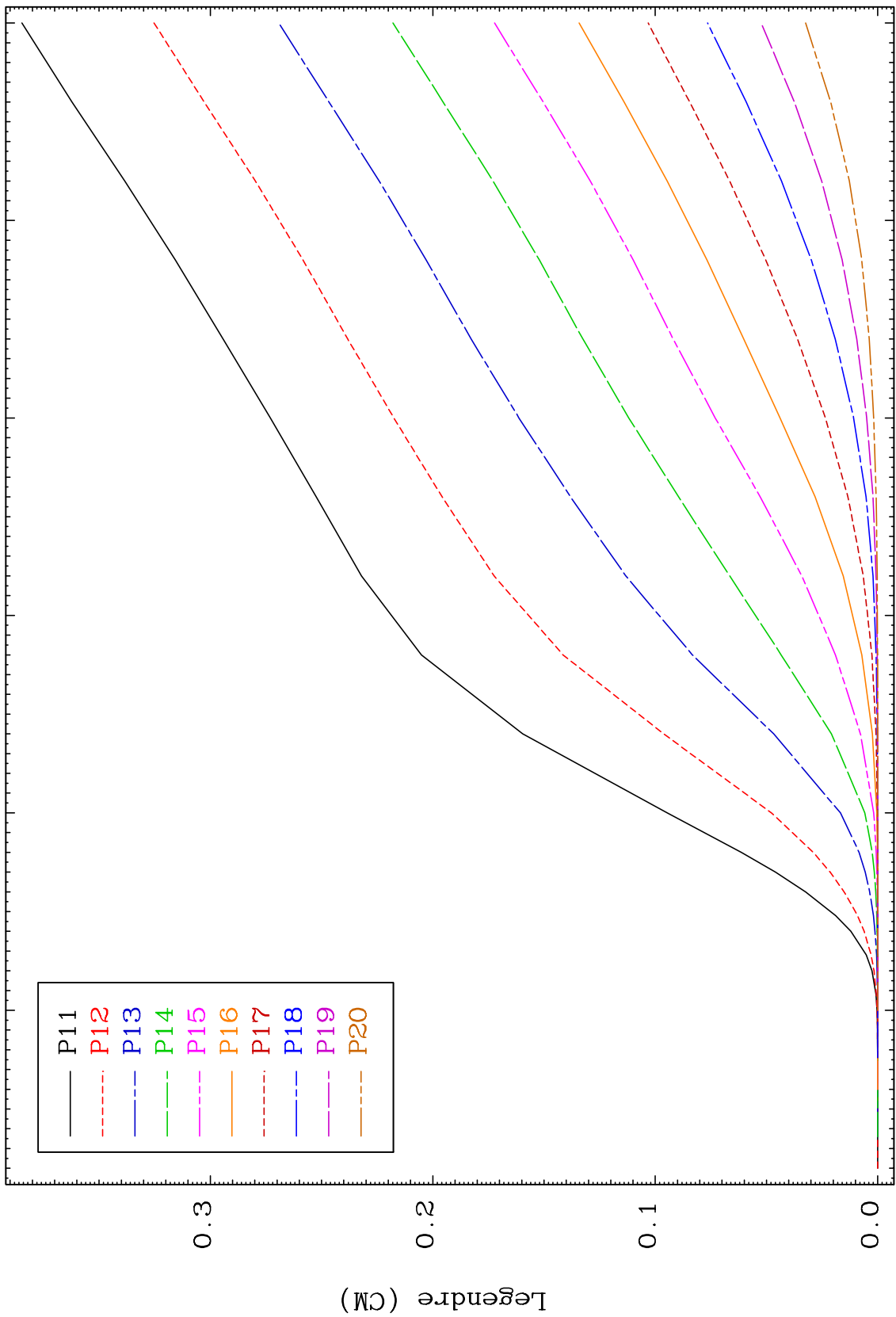
92-U -227



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Elastic Legendre Coefficients

92-U -227



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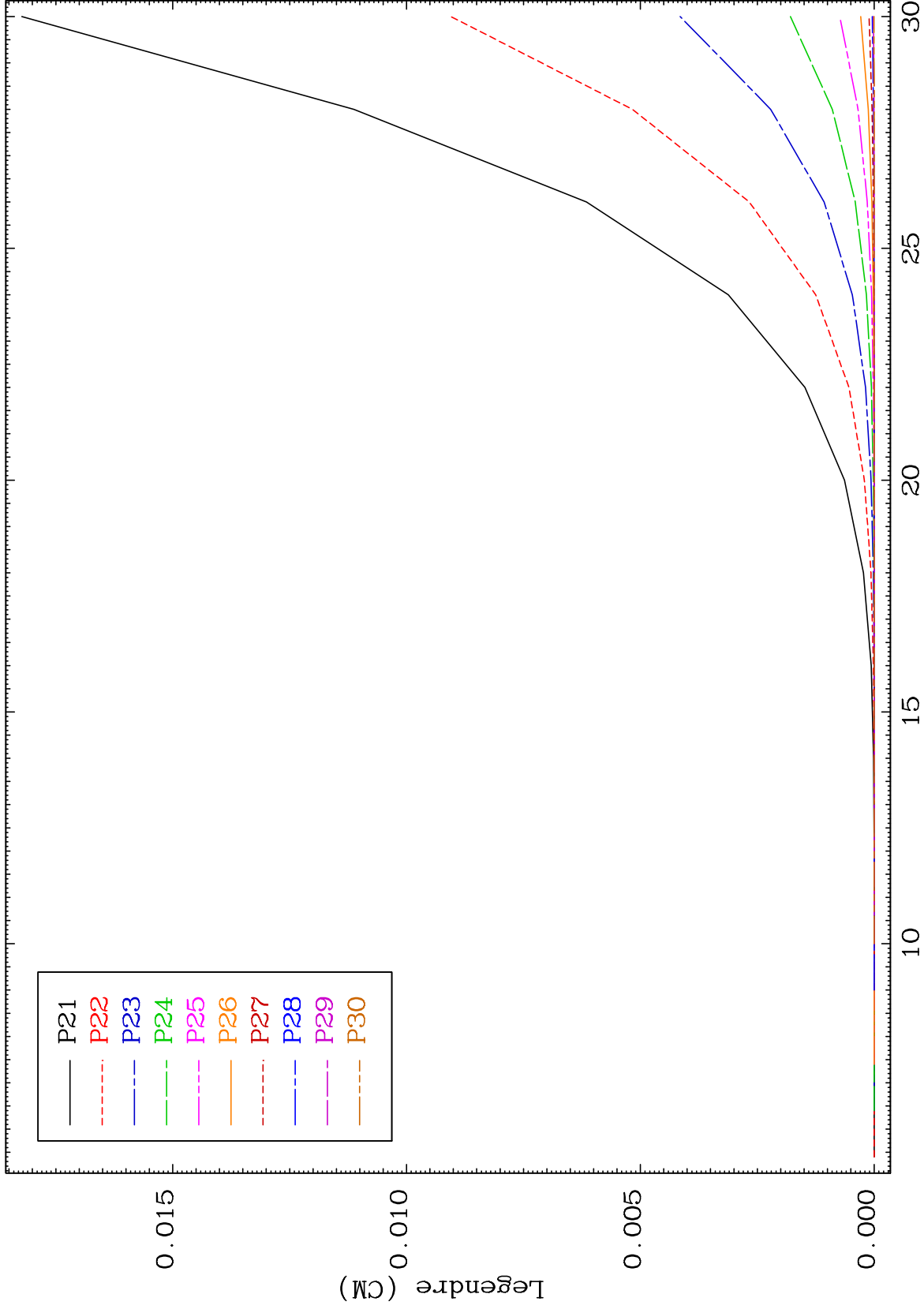
Incident Energy (MeV)

92-U -227

MAT 9204

Elastic Legendre Coefficients

92-U -227



22

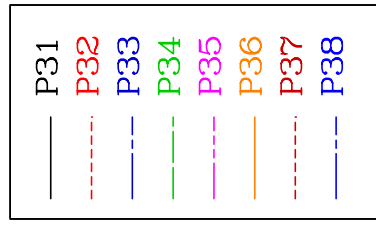
Incident Energy (MeV)

92-U -227

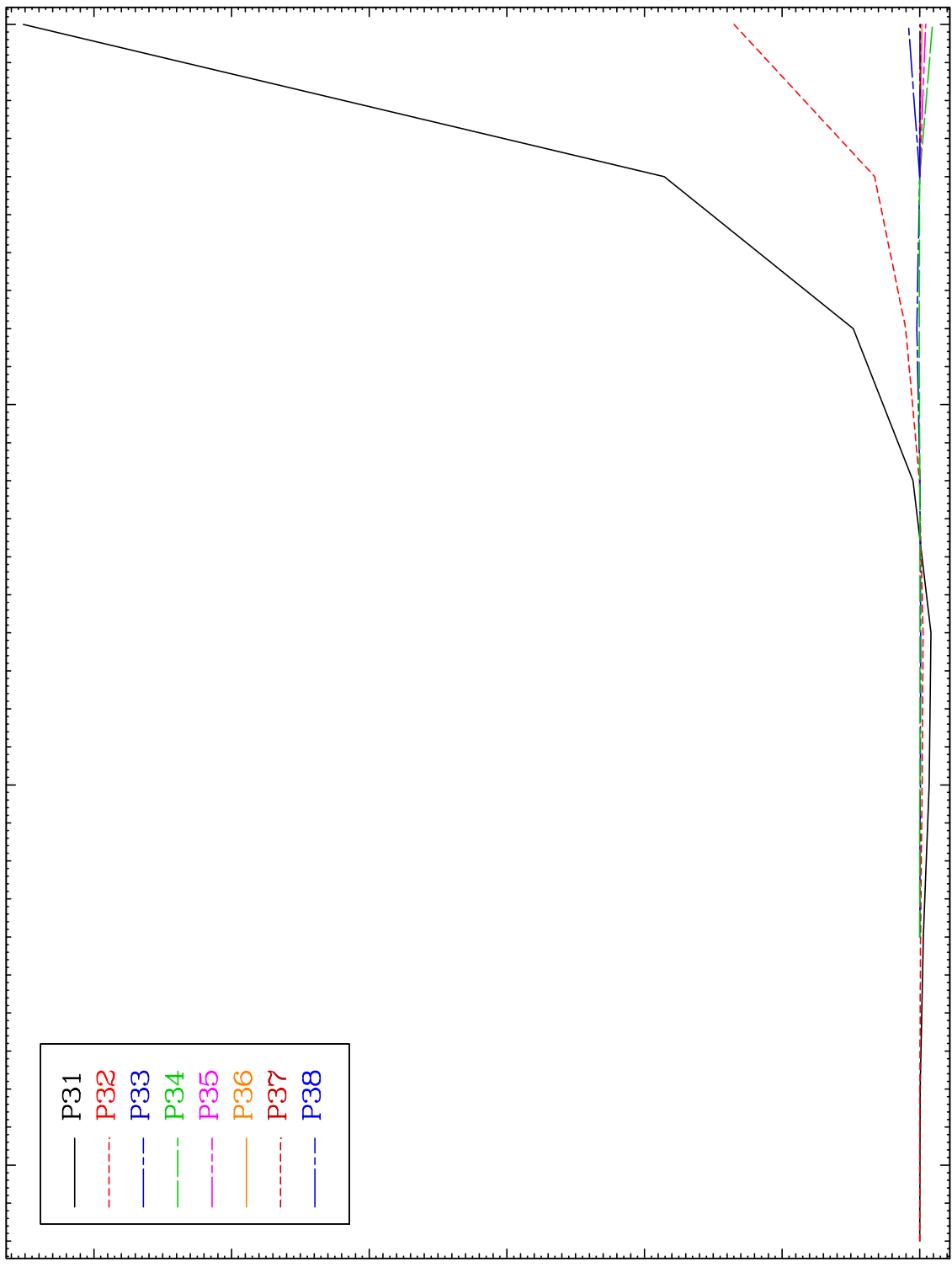
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Elastic
Legendre Coefficients

92-U -227



Legendre (CM)
 $\times 10^{-6}$



23

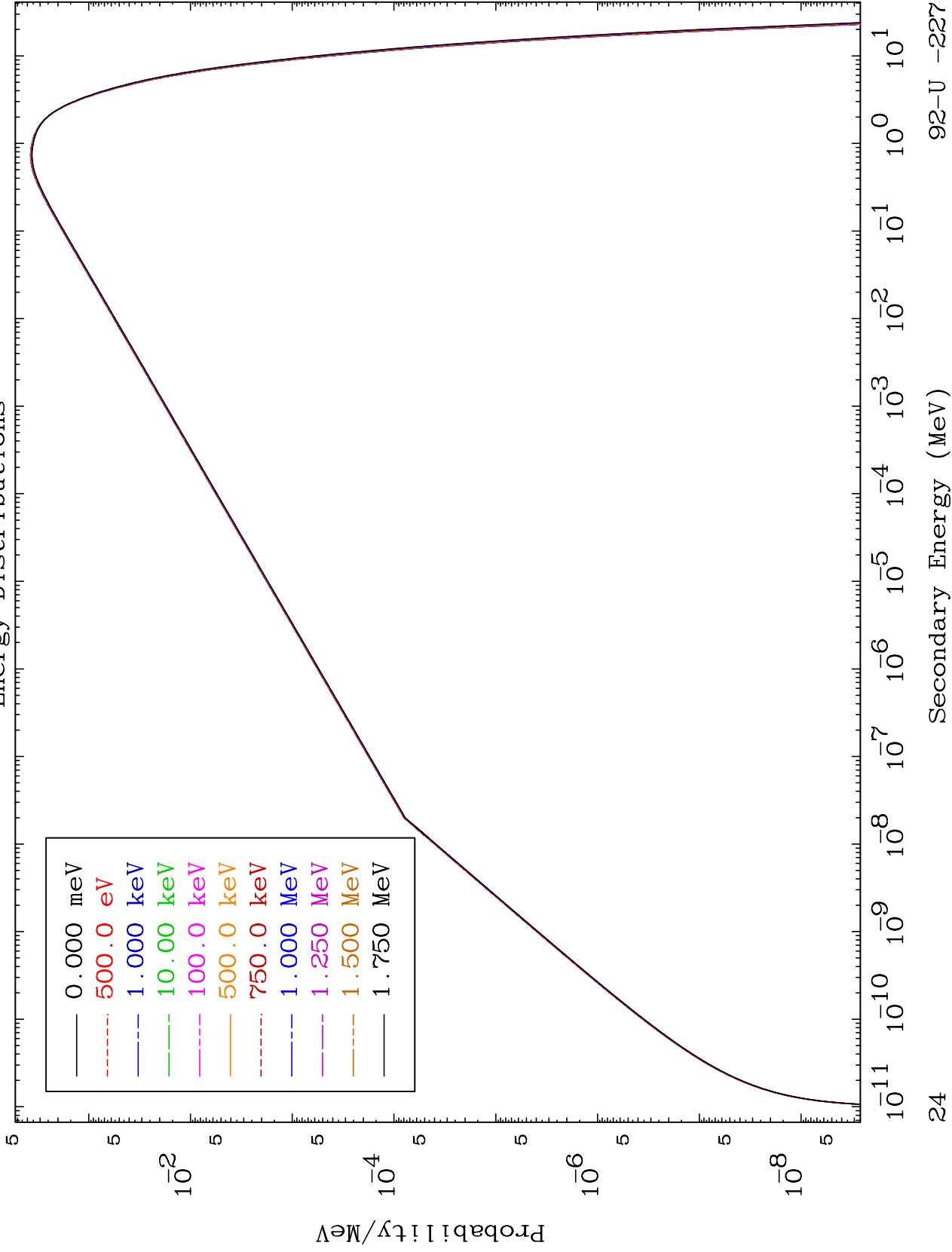
Incident Energy (MeV)

92-U -227

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Fission Energy Distributions

92-U -227



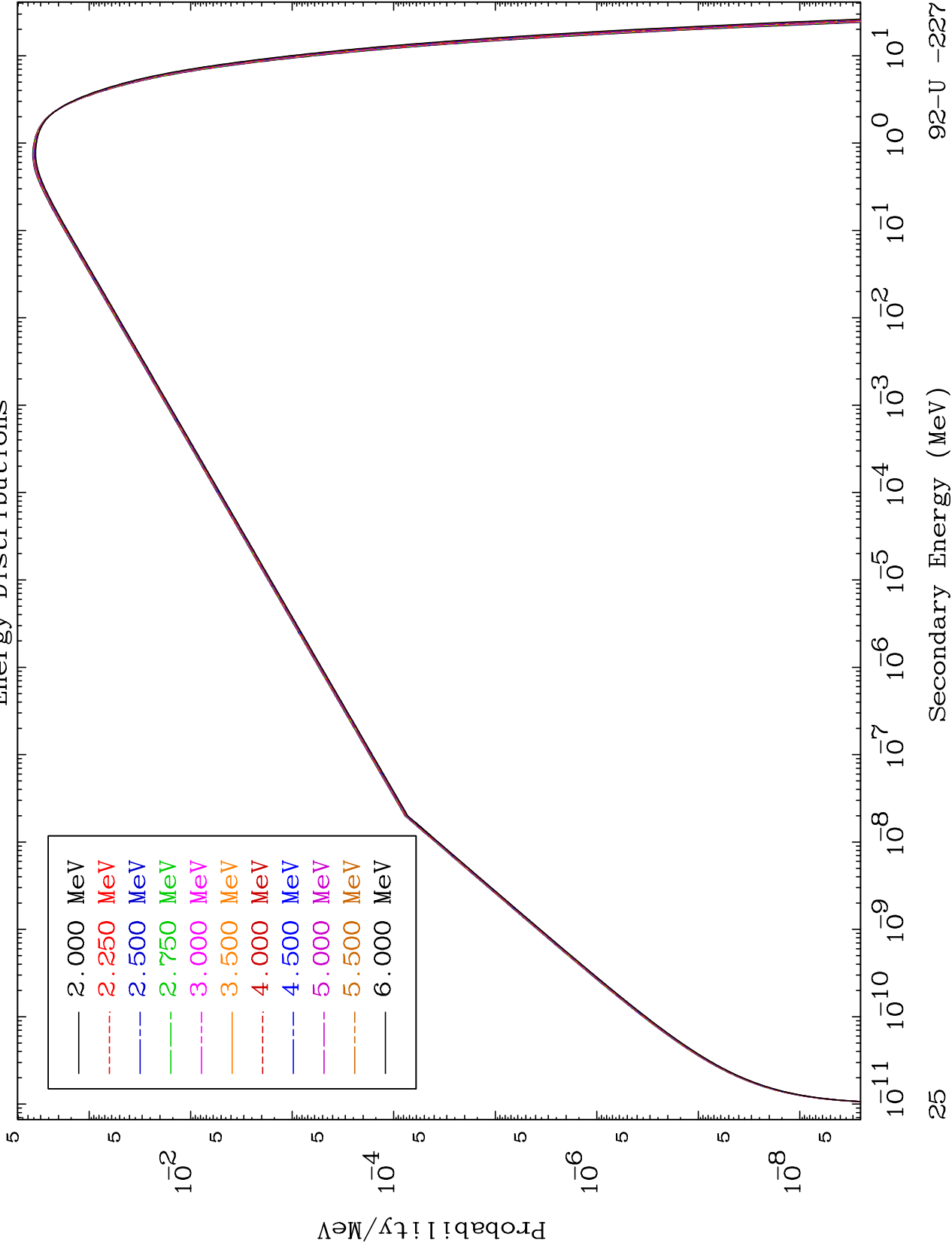
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Fission Energy Distributions

92-U -227

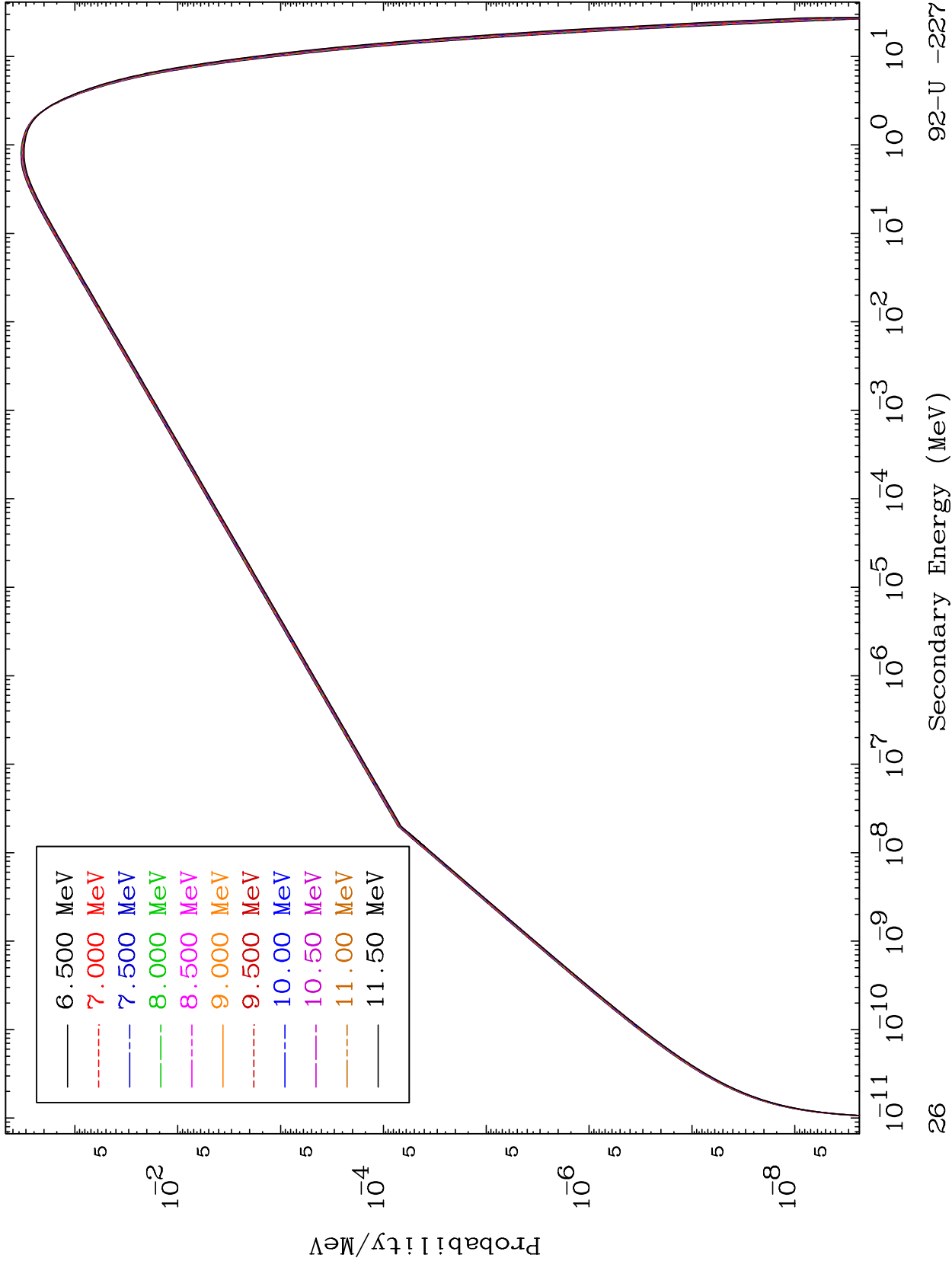


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Secondary Energy (MeV)

92-U -227

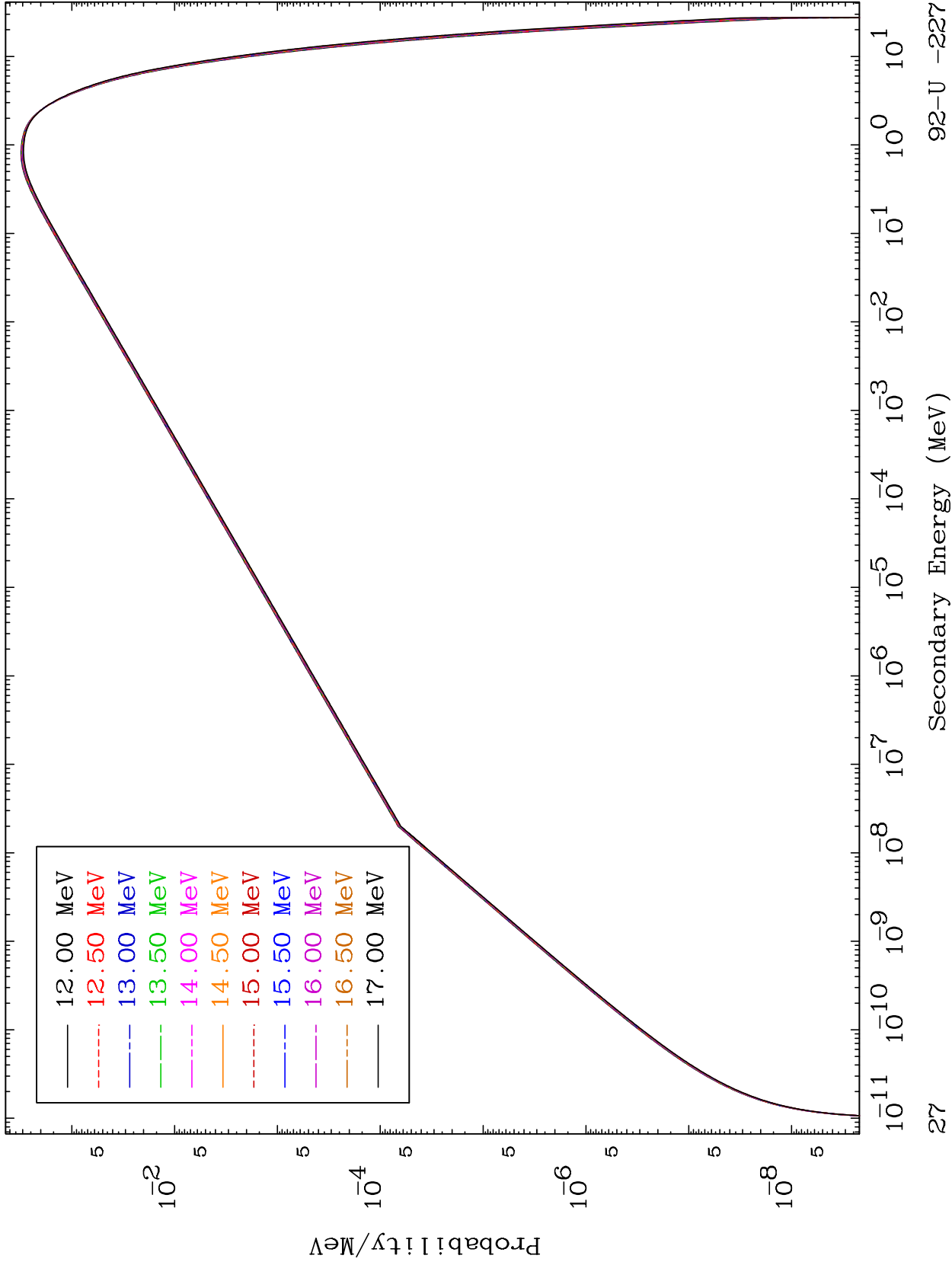
Fission Energy Distributions



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Fission Energy Distributions

92-U -227

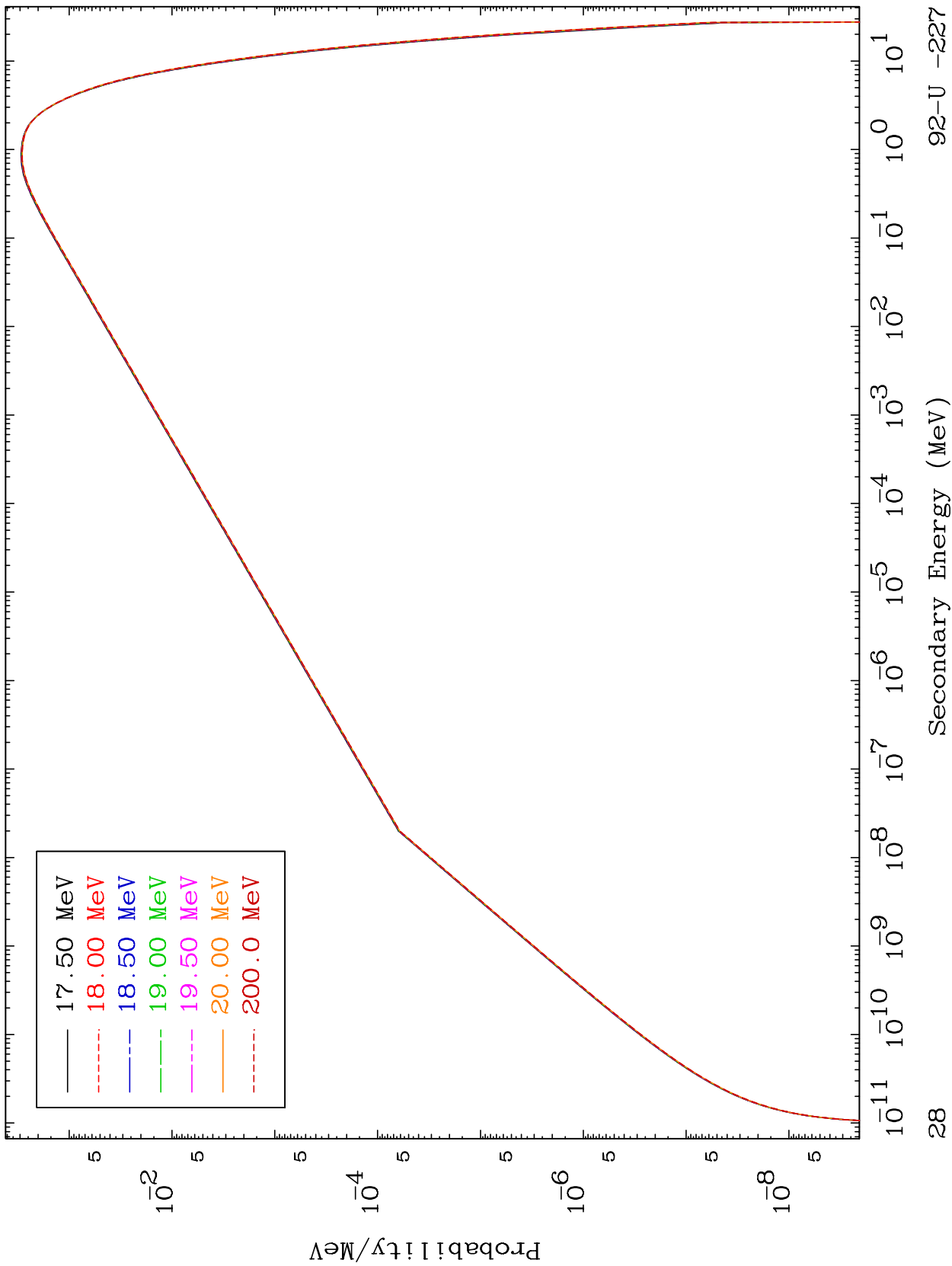


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Secondary Energy (MeV)

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Fission Energy Distributions

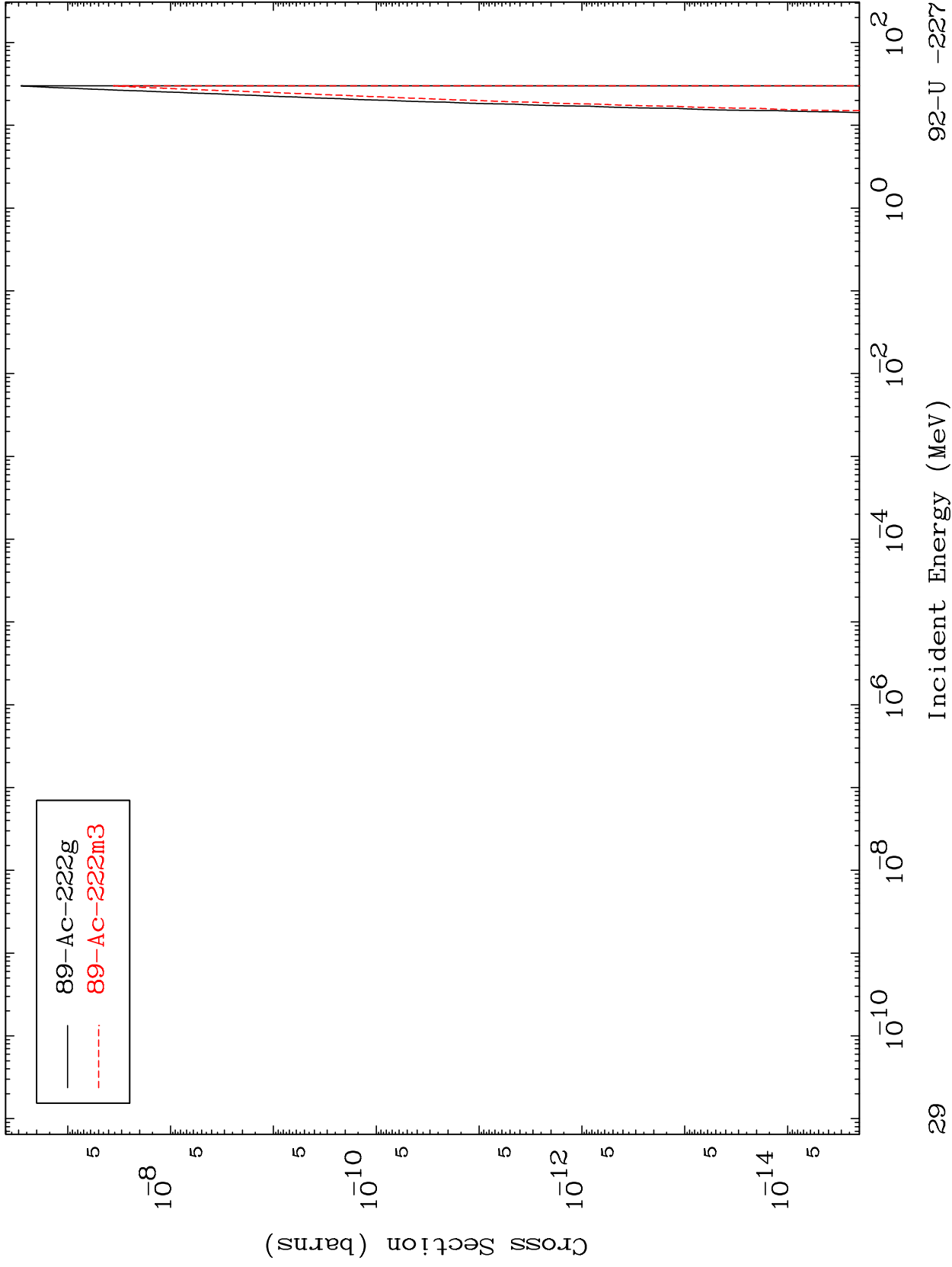


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(n,n') p α

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Radionuclide Production Cross Section



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Incident Energy (MeV)

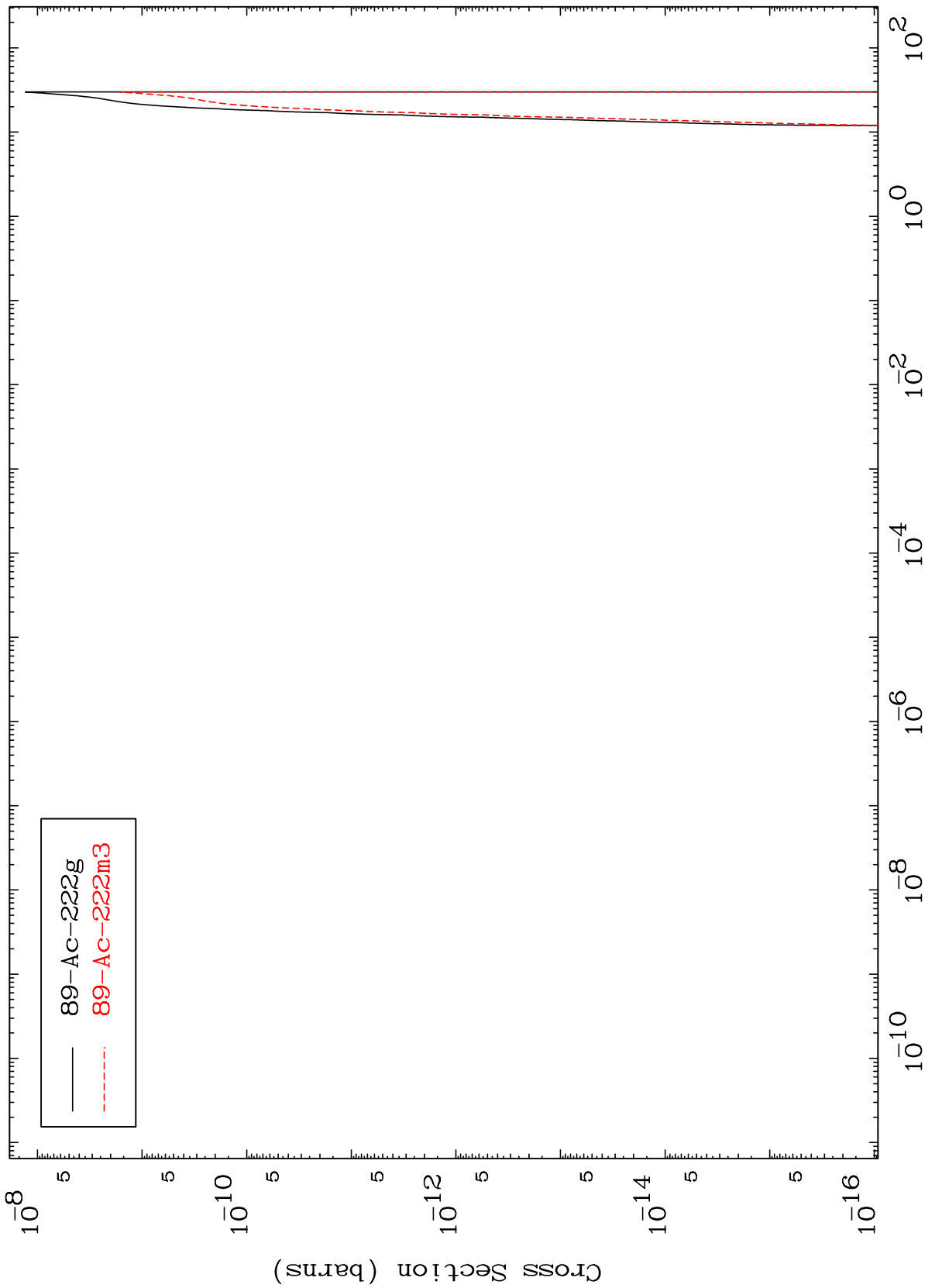
92-U -227

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(n,d) α

92-U -227

Radionuclide Production Cross Section



89-Ac-222g
89-Ac-222m3

Incident Energy (MeV)

92-U -227

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