

Program EVALPLOT
(Version 2021-1)

by

Dermott E. Cullen
(Present Contact Information)
Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

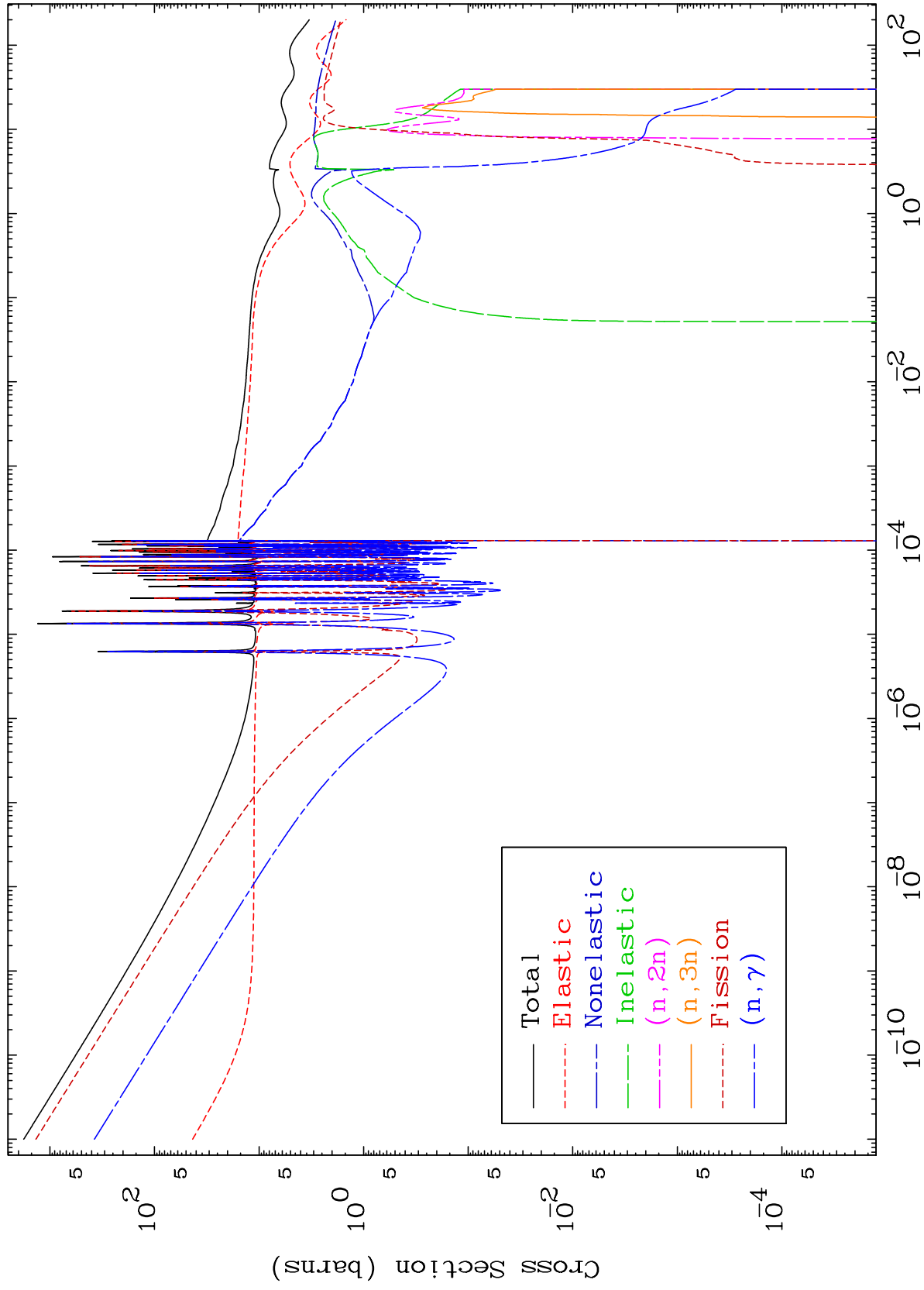
E.Mail: redcullen1@comcast.net
Web: redcullen1.net/HOMEPAGE.NEW

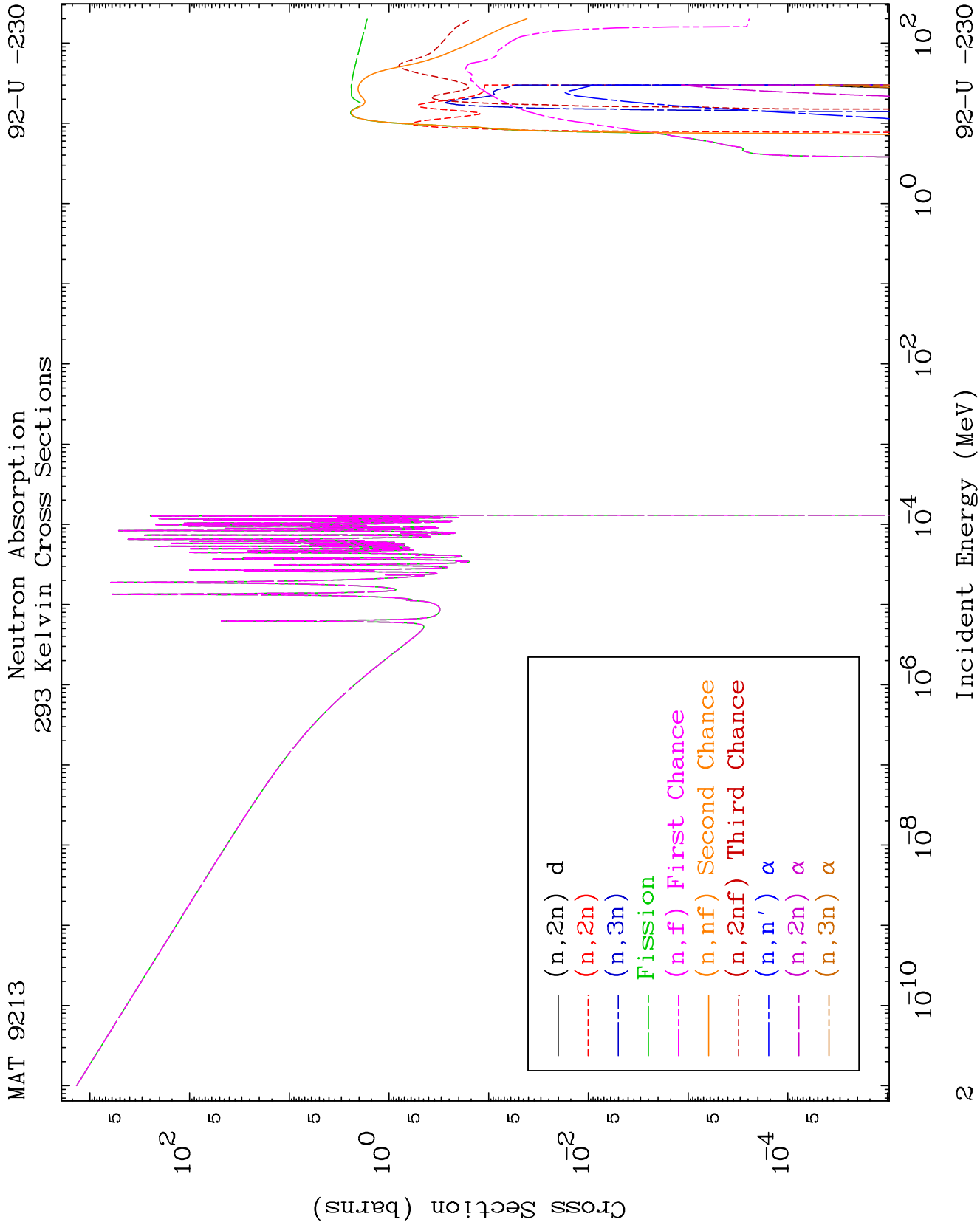
Press Mouse Button to Start

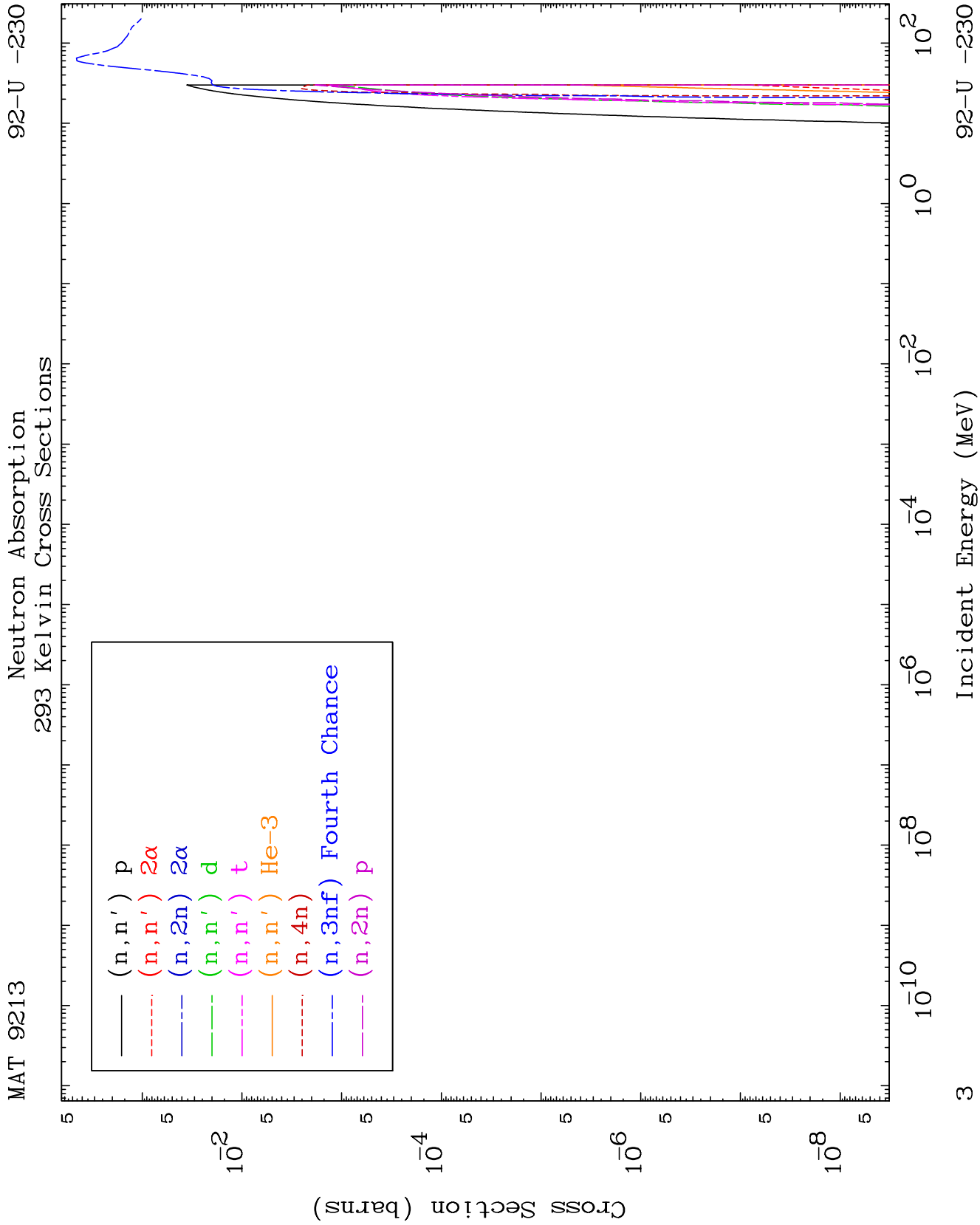
MAT 9213

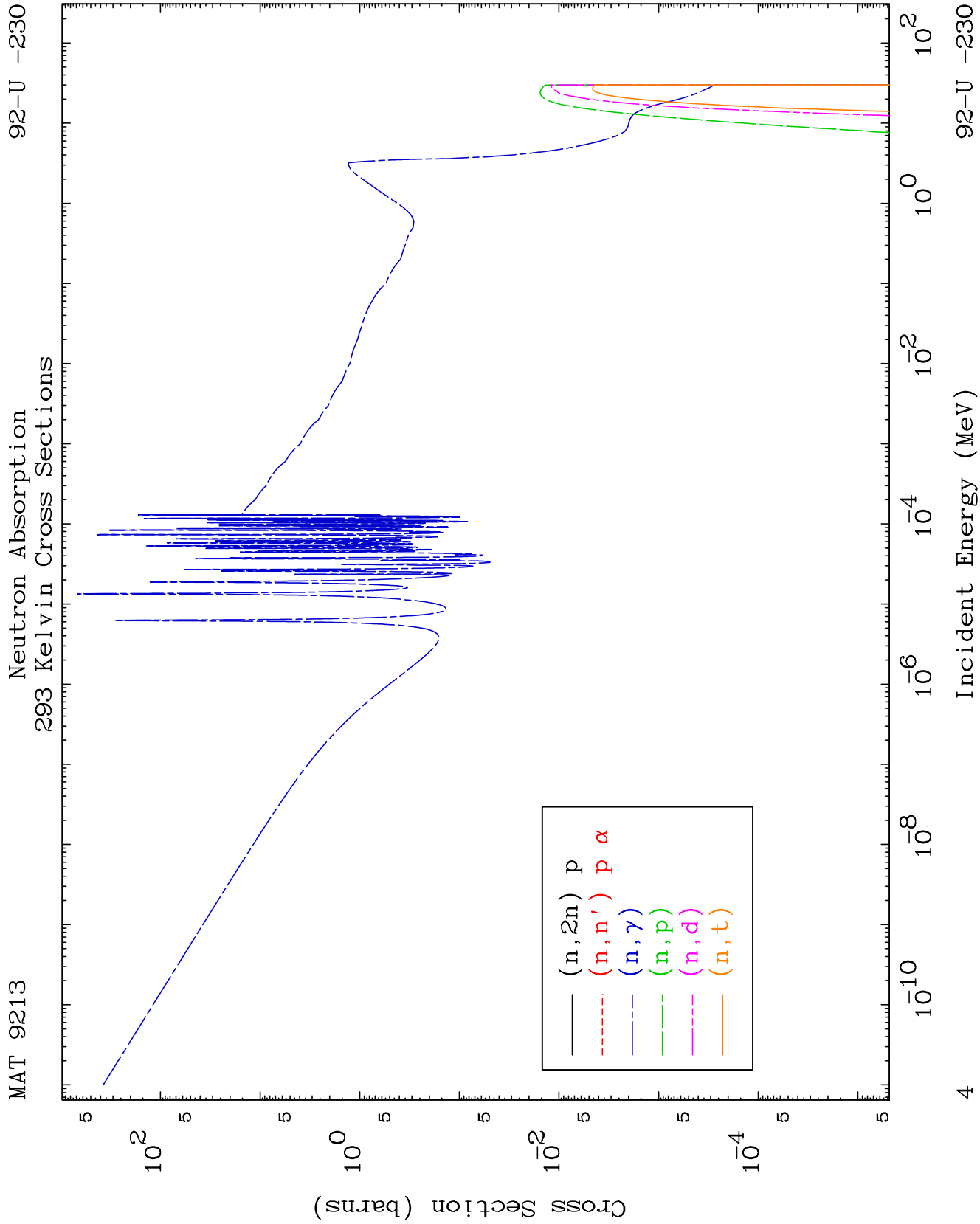
Neutron Major
293 Kelvin Cross Sections

92-U -230





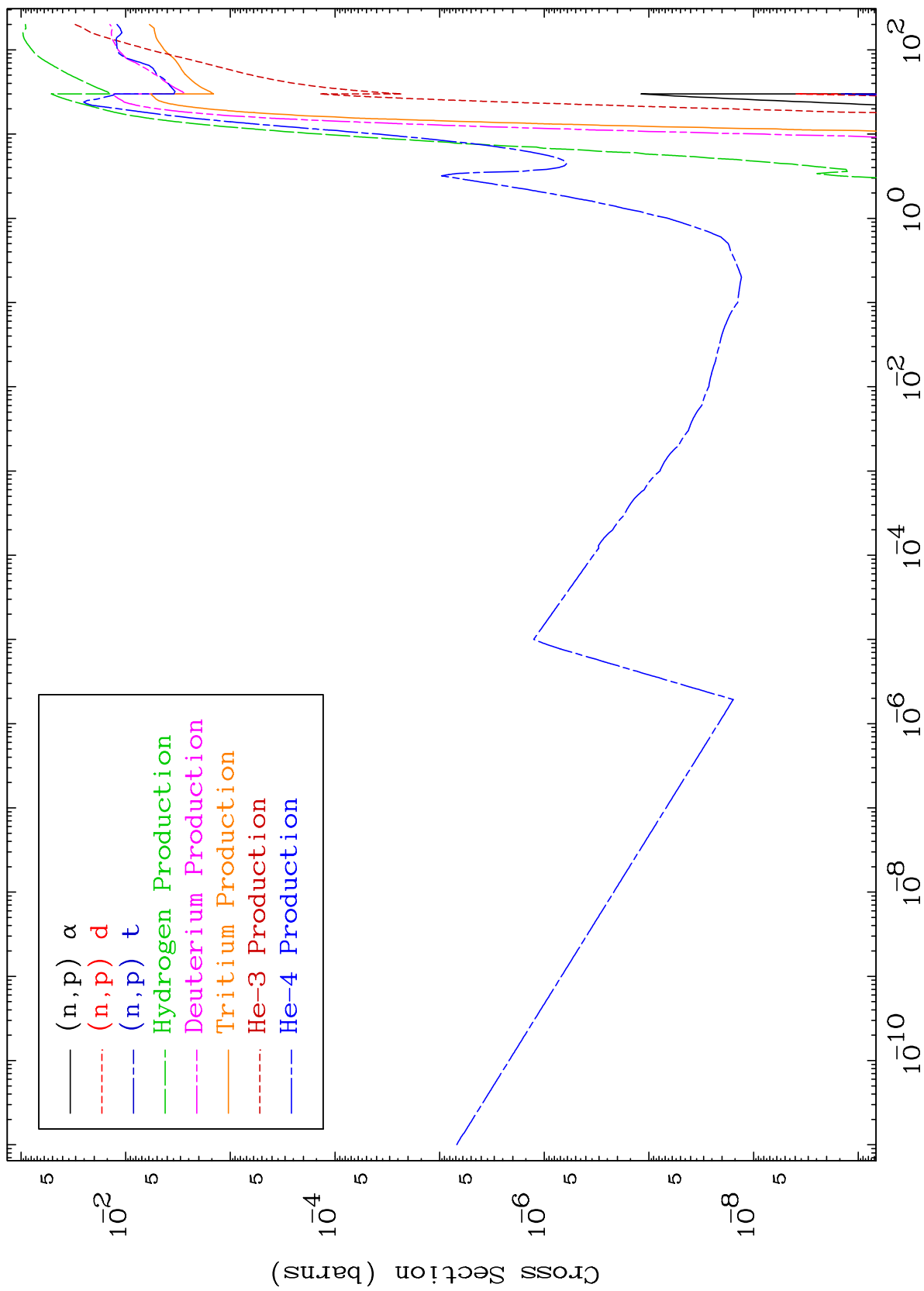




MAT 9213

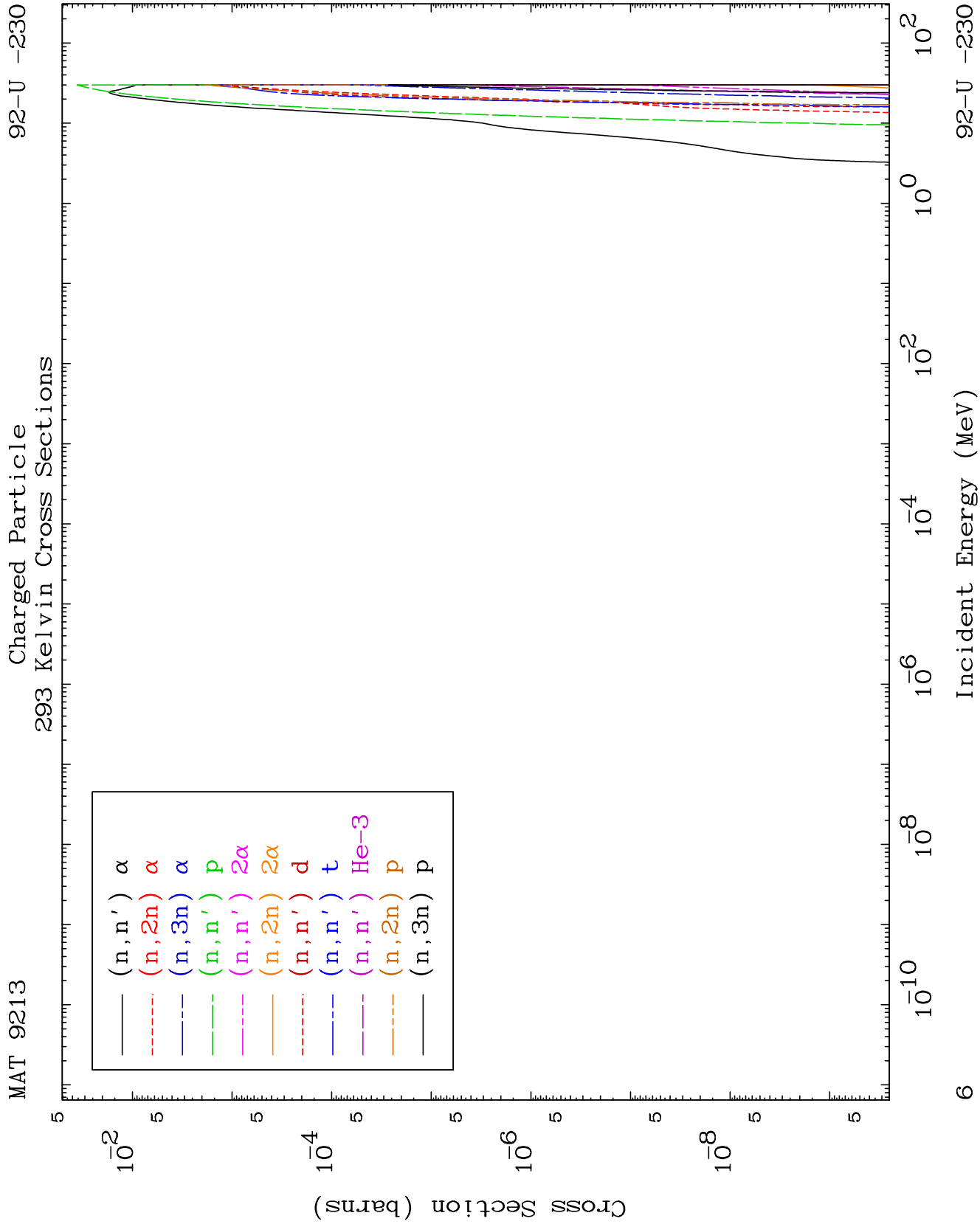
Neutron Absorption
293 Kelvin Cross Sections

92-U -230



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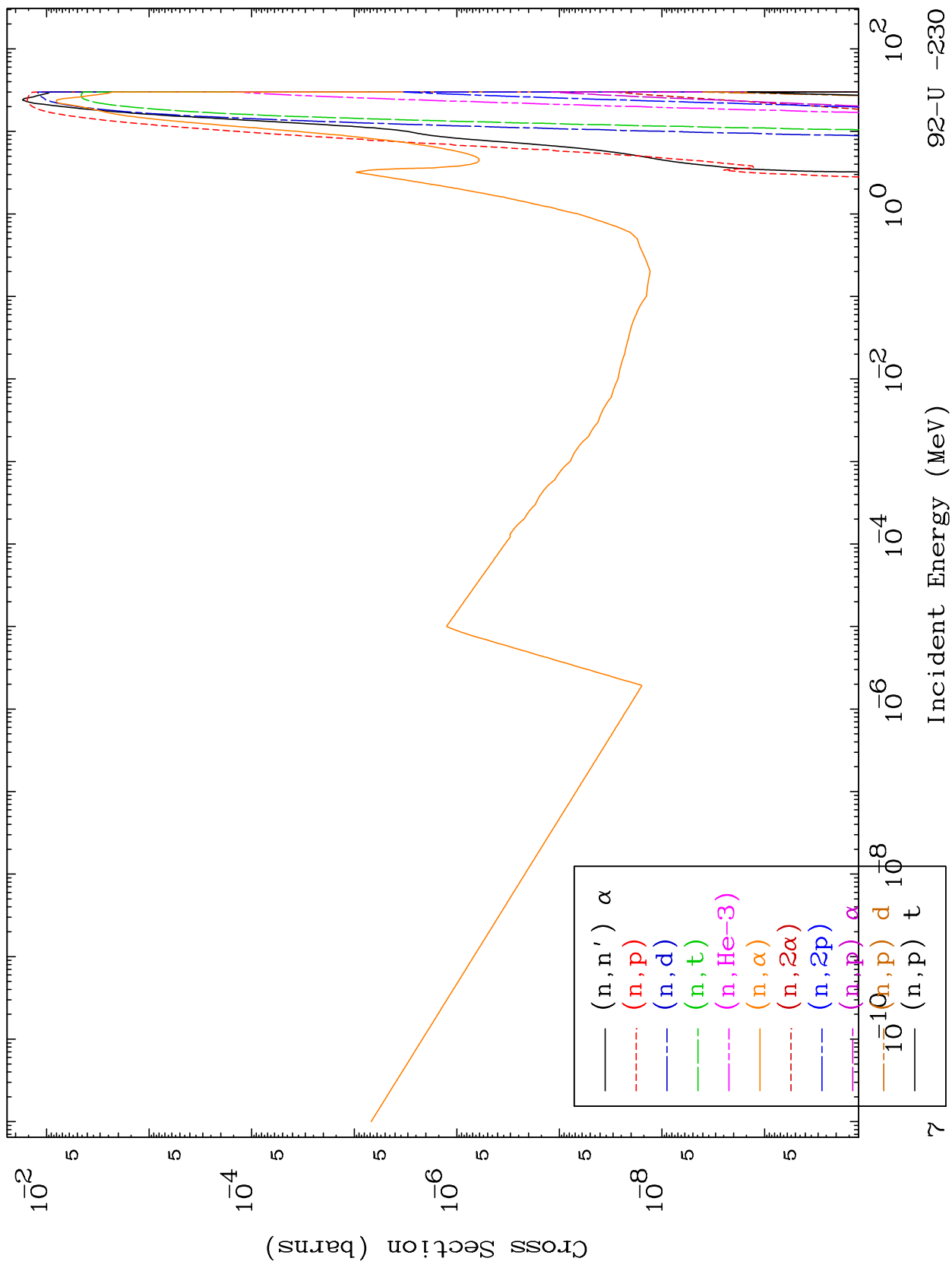
92-U -230



MAT 9213

Charged Particle 293 Kelvin Cross Sections

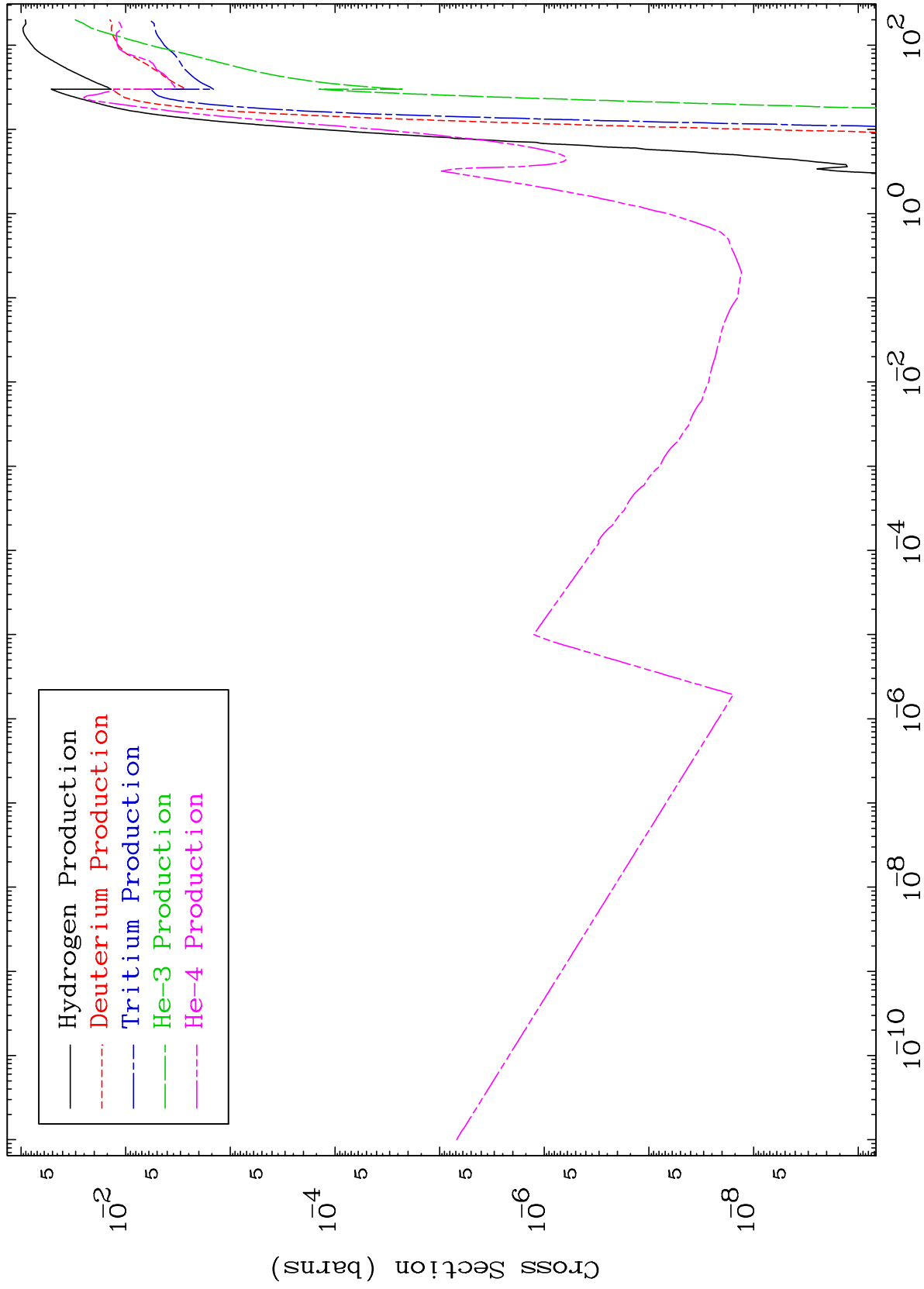
92-U -230



MAT 9213

Particle Production
293 Kelvin Cross Sections

92-U -230



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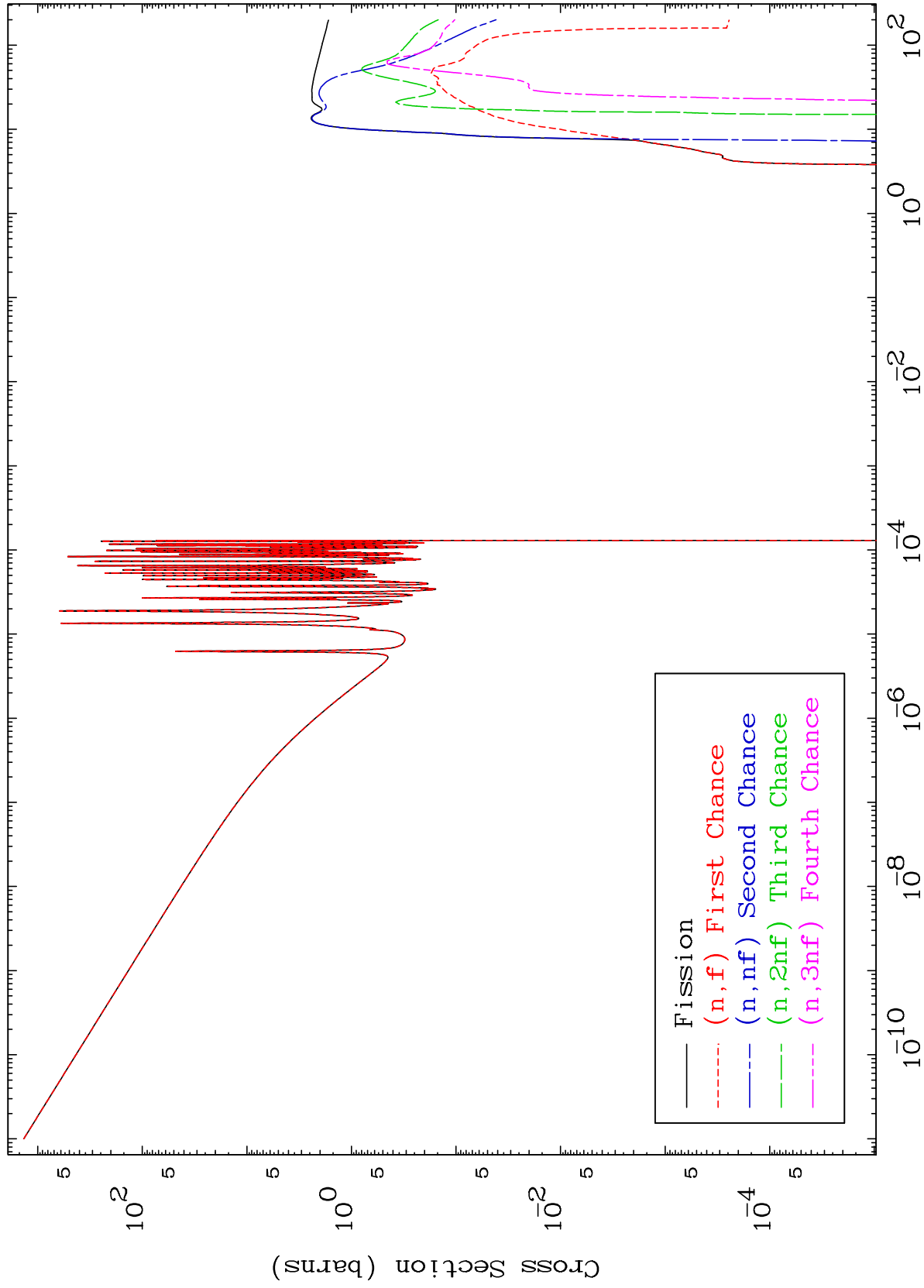
Incident Energy (MeV)

92-U -230

MAT 9213

Fission
293 Kelvin Cross Sections

92-U -230

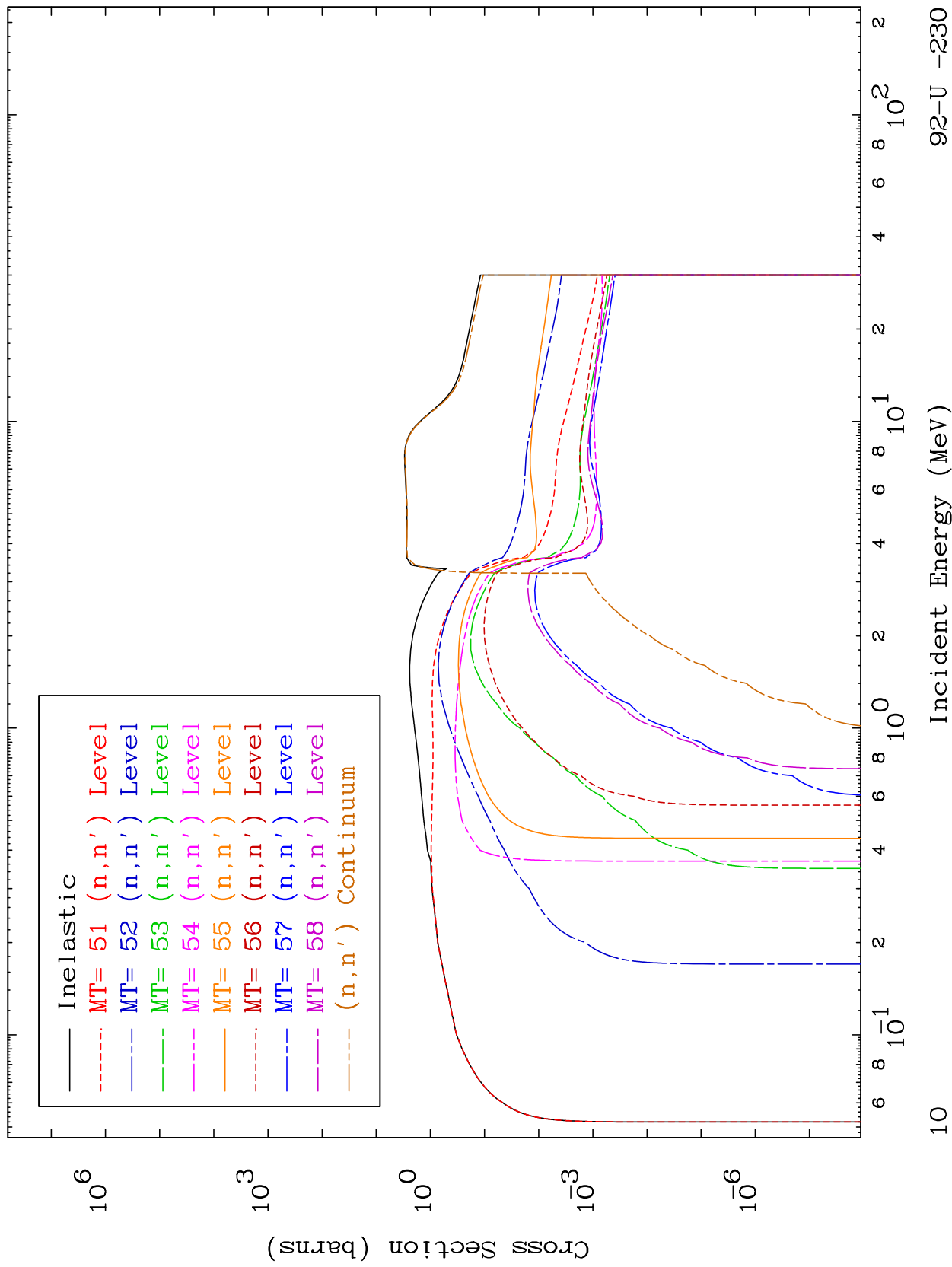


MAT 9213

(n,n') Levels

293 Kelvin Cross Sections

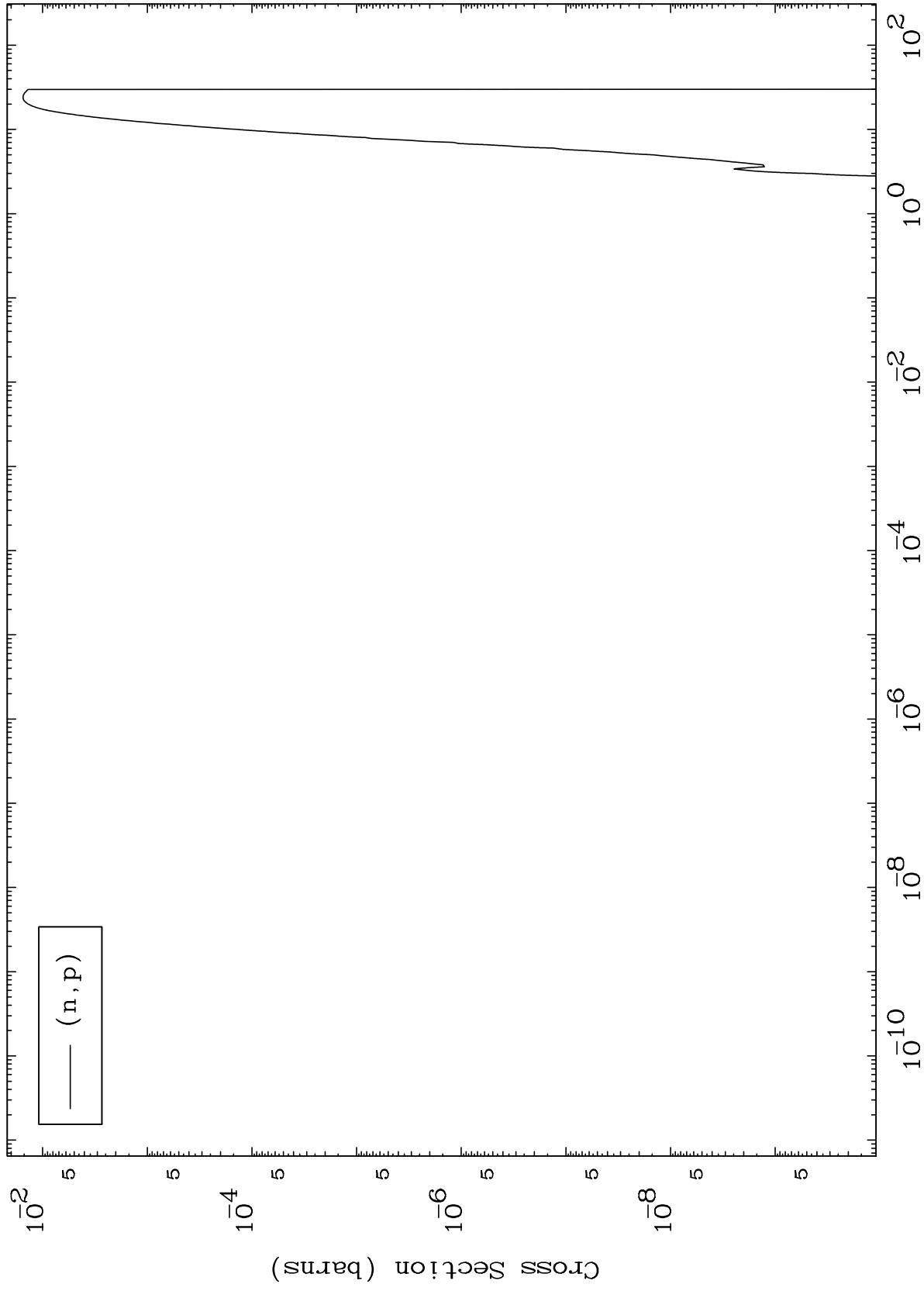
92-U -230

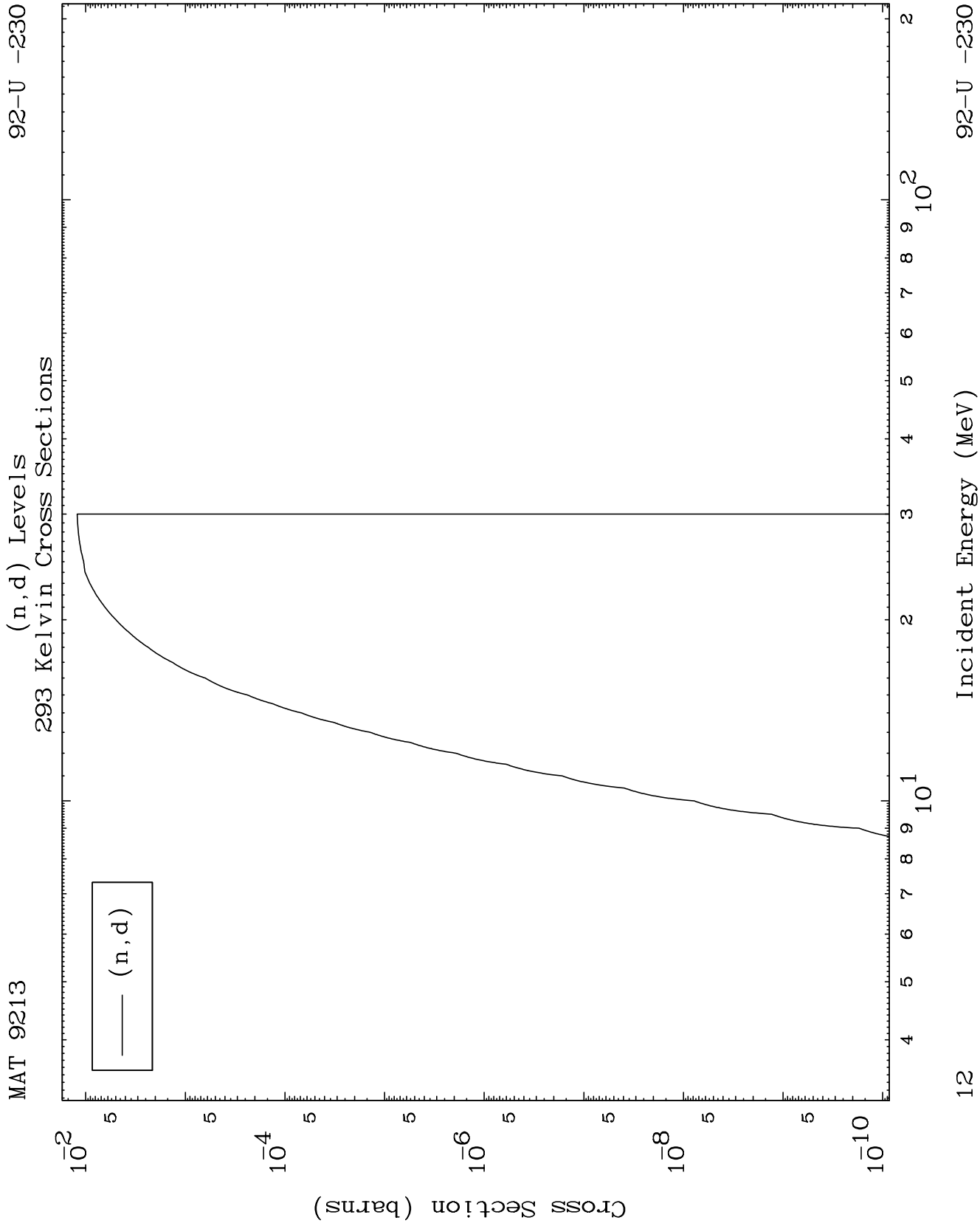


MAT 9213

(n,p) Levels
293 Kelvin Cross Sections

92-U -230

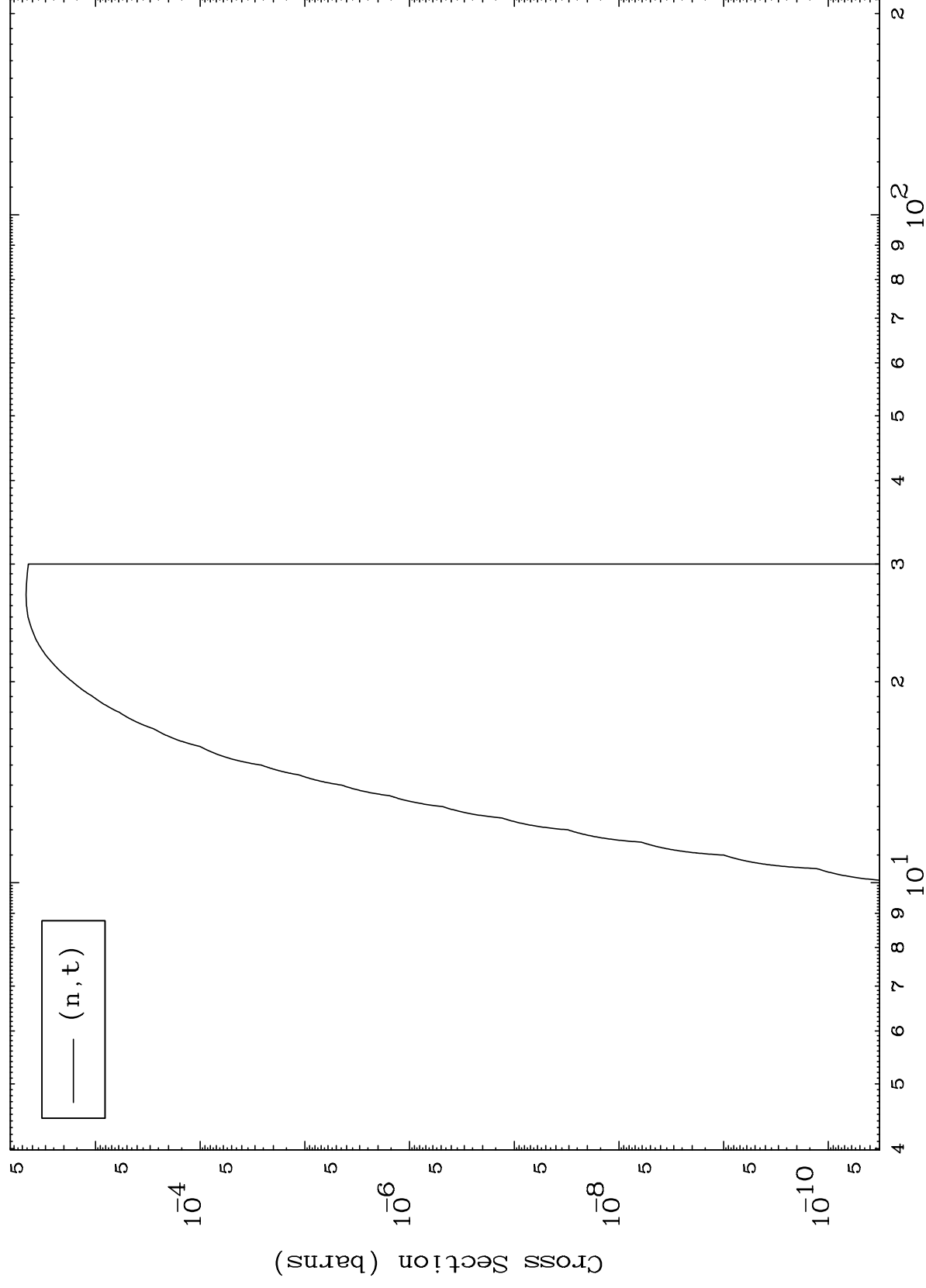




MAT 9213

(n, t) Levels
293 Kelvin Cross Sections

92-U -230



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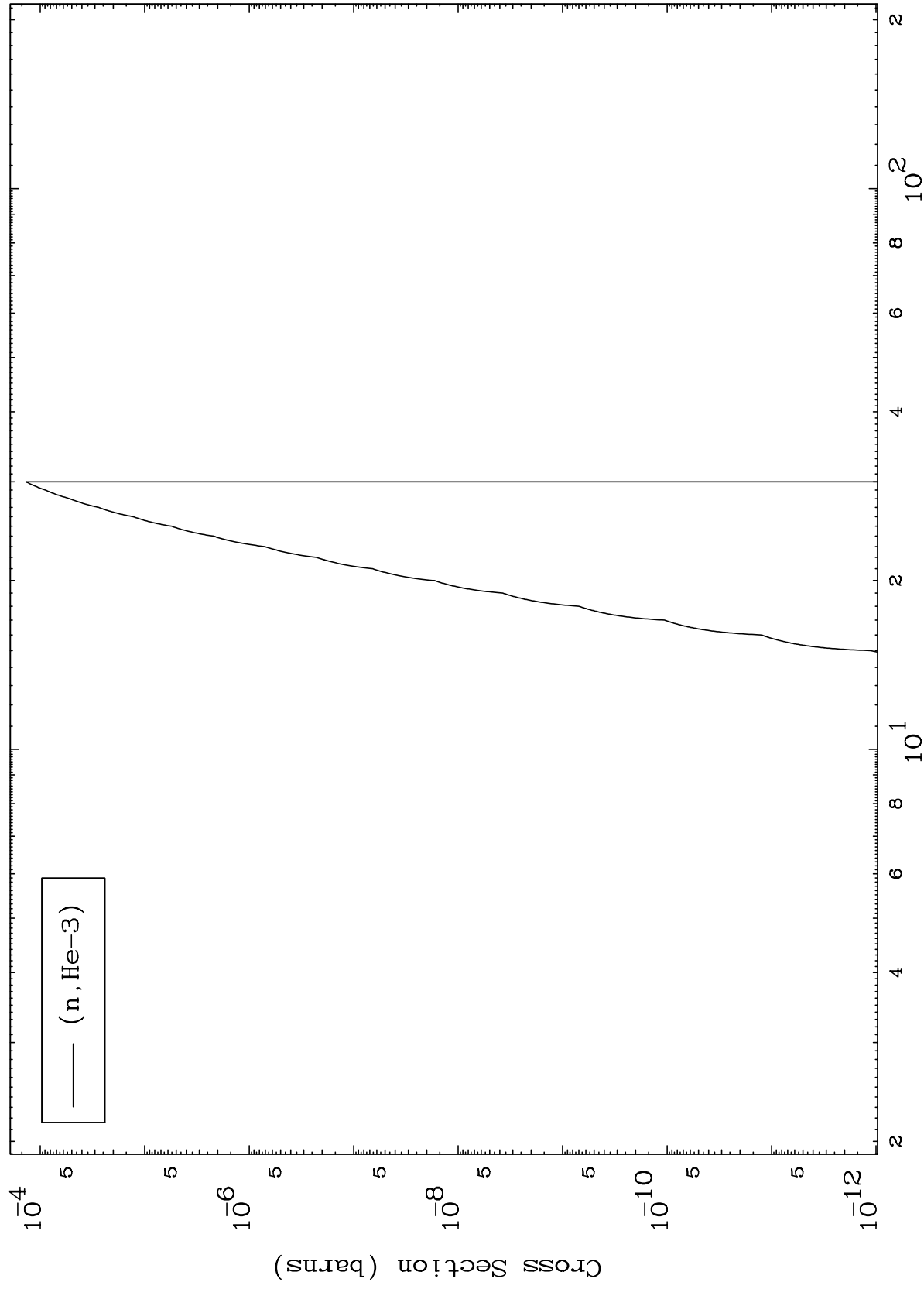
Incident Energy (MeV)

92-U -230

MAT 9213

(n,He3) Levels
293 Kelvin Cross Sections

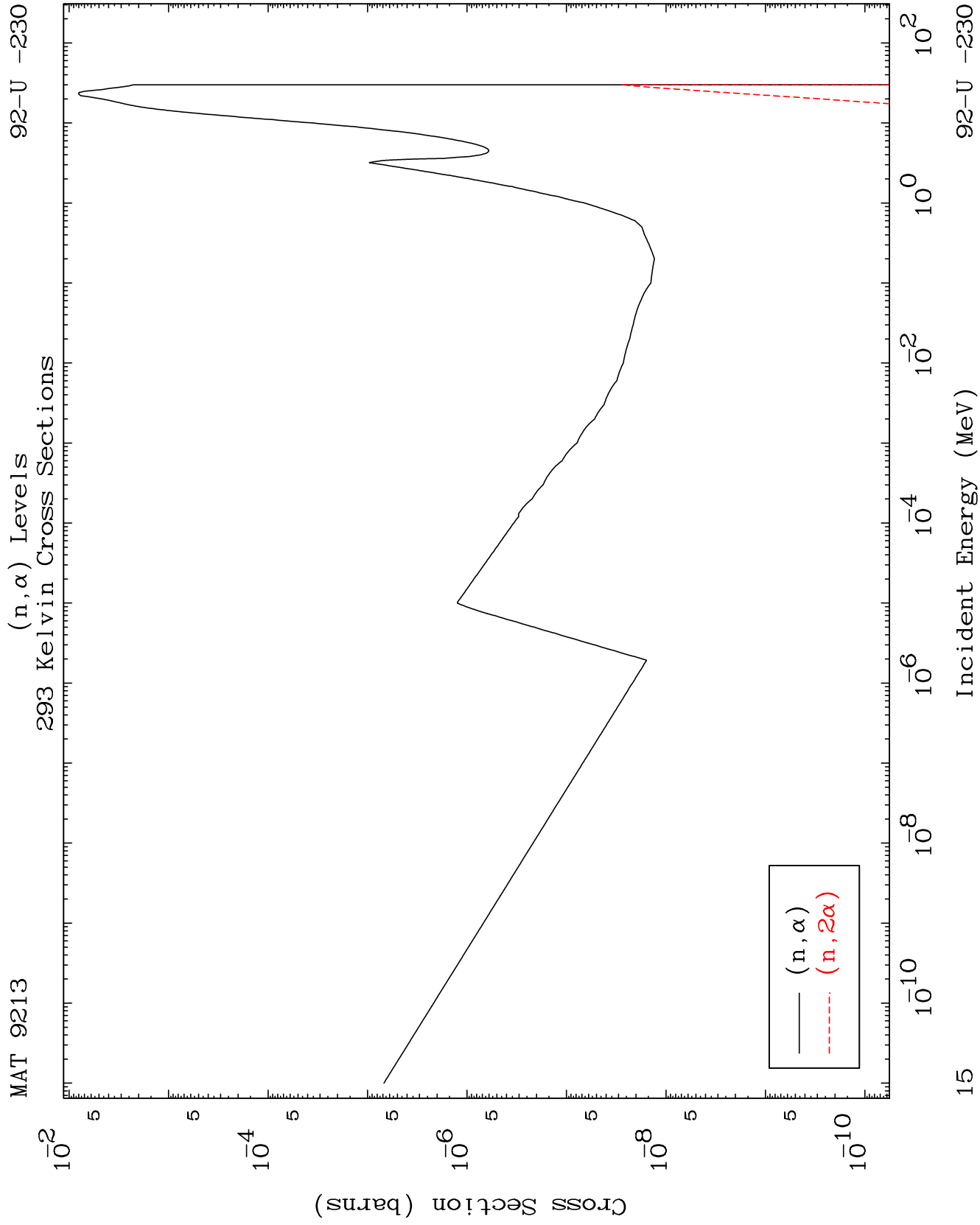
92-U -230



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Incident Energy (MeV)

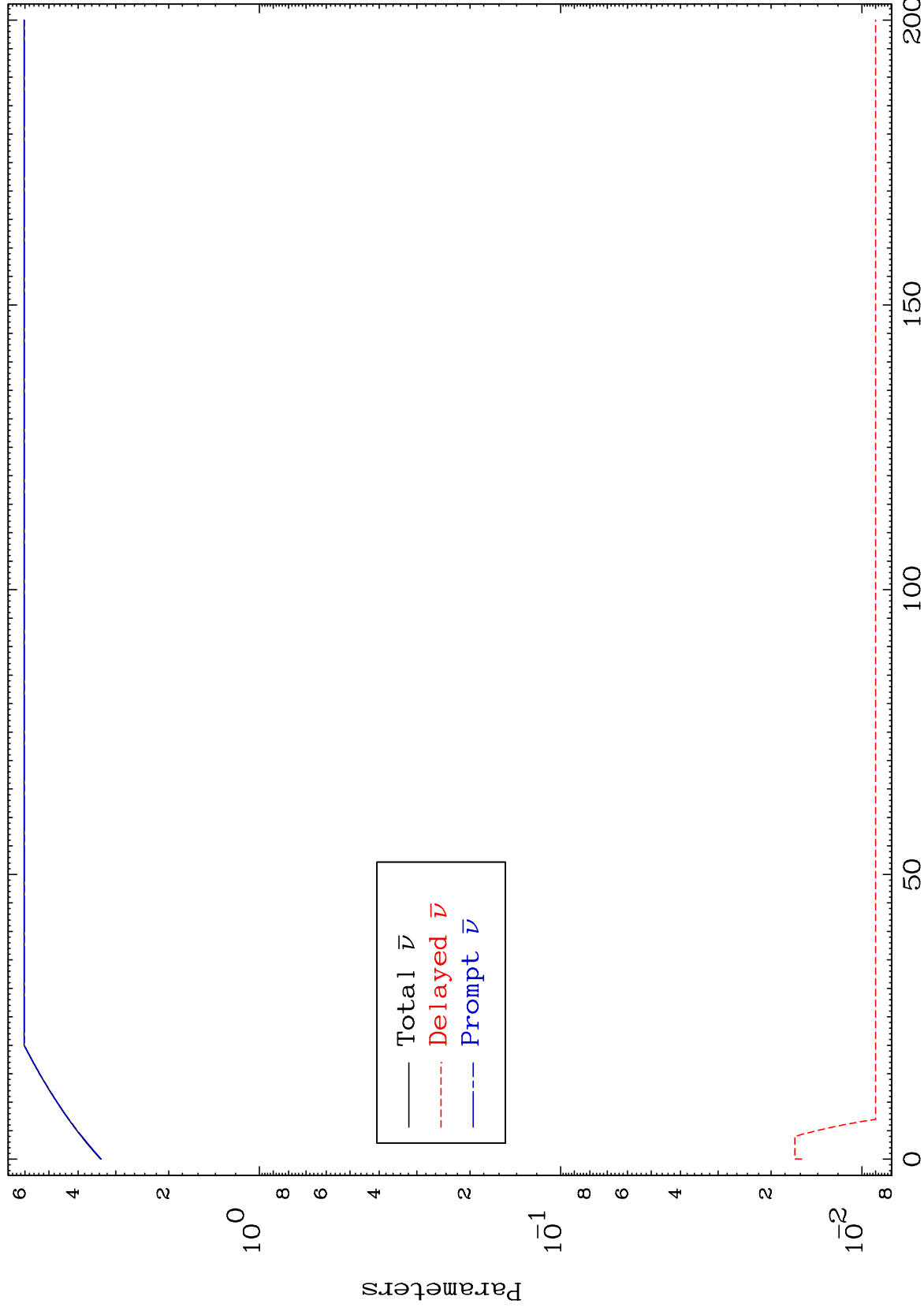
92-U -230



MAT 9213

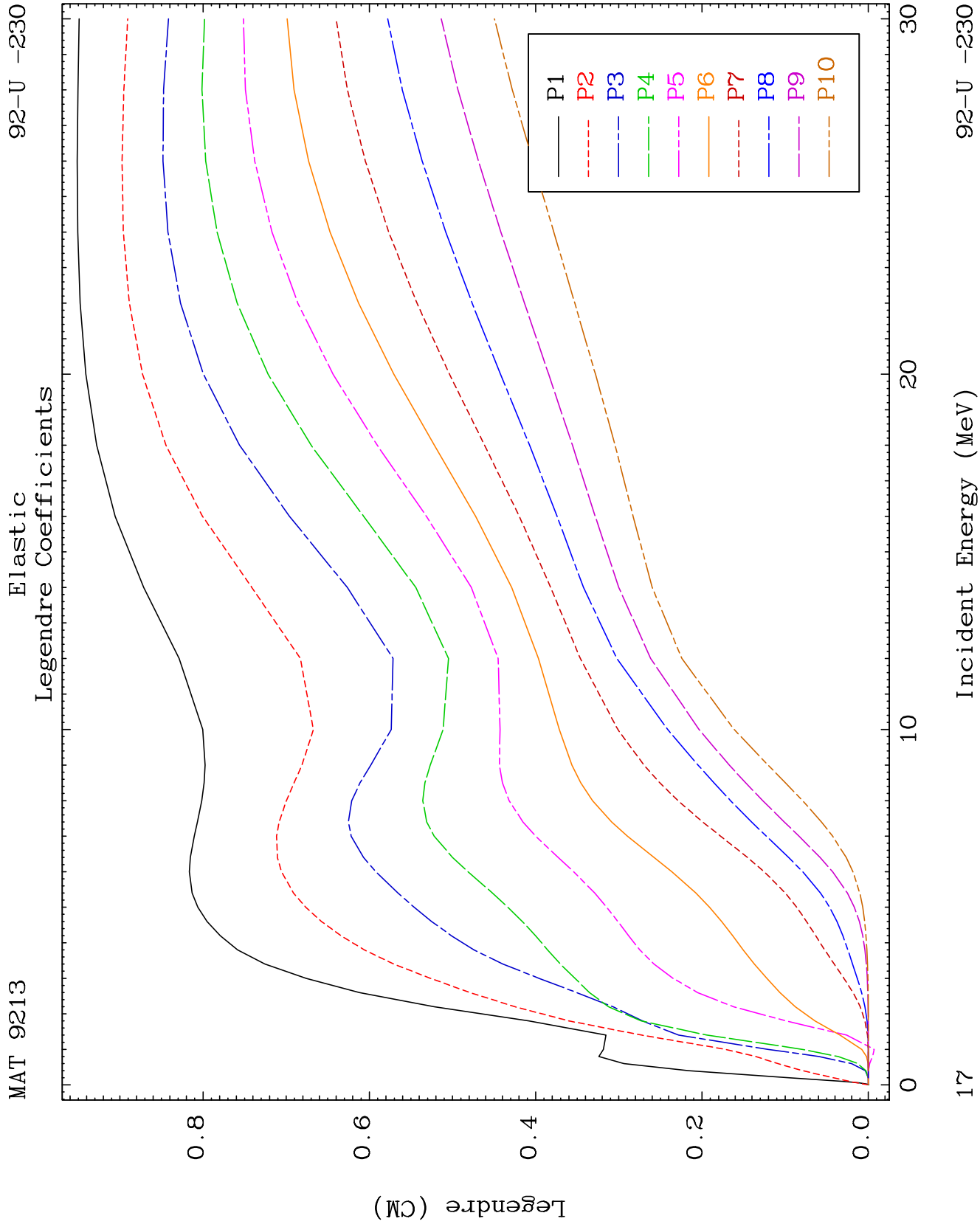
Energy Release
Parameters

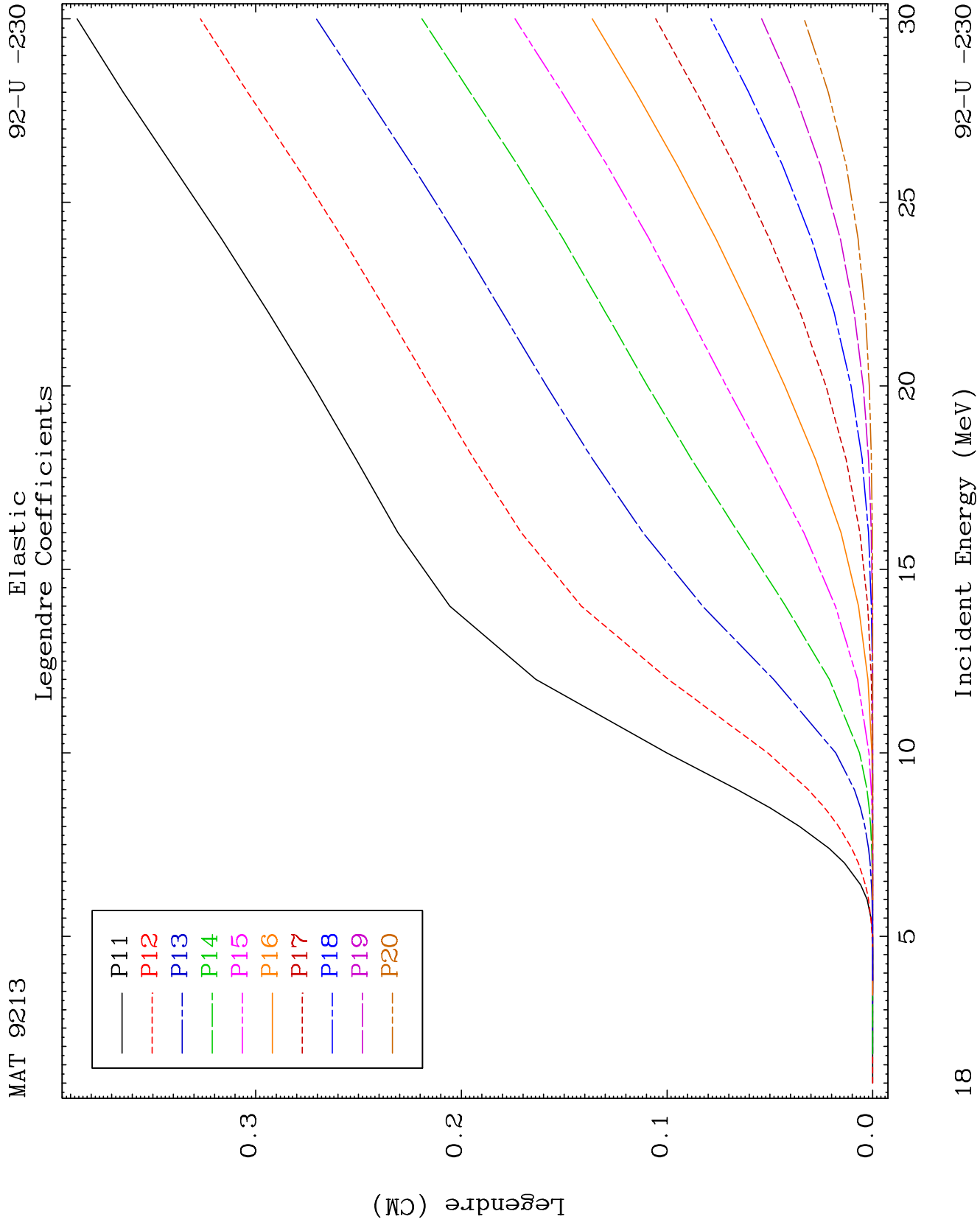
92-U -230

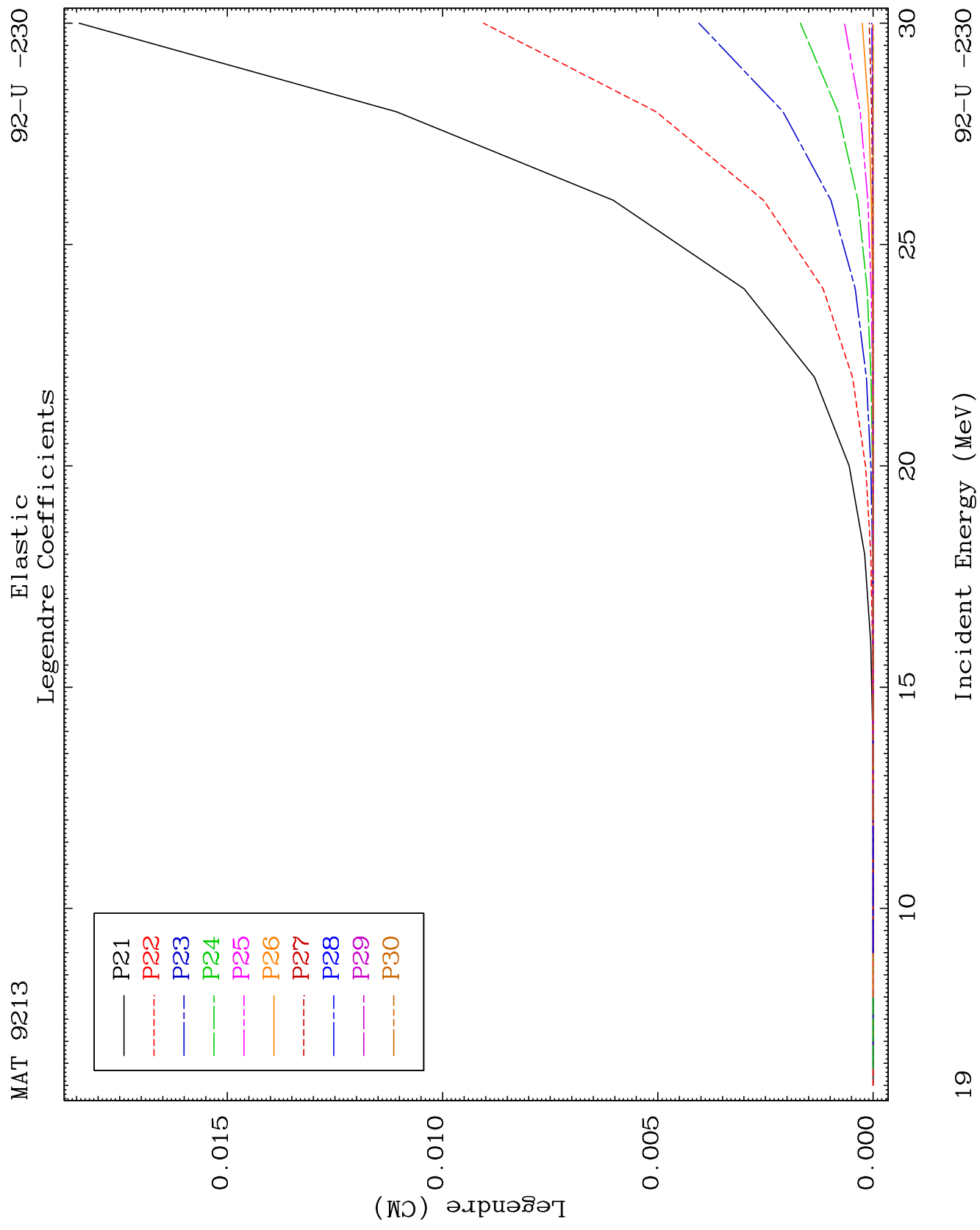


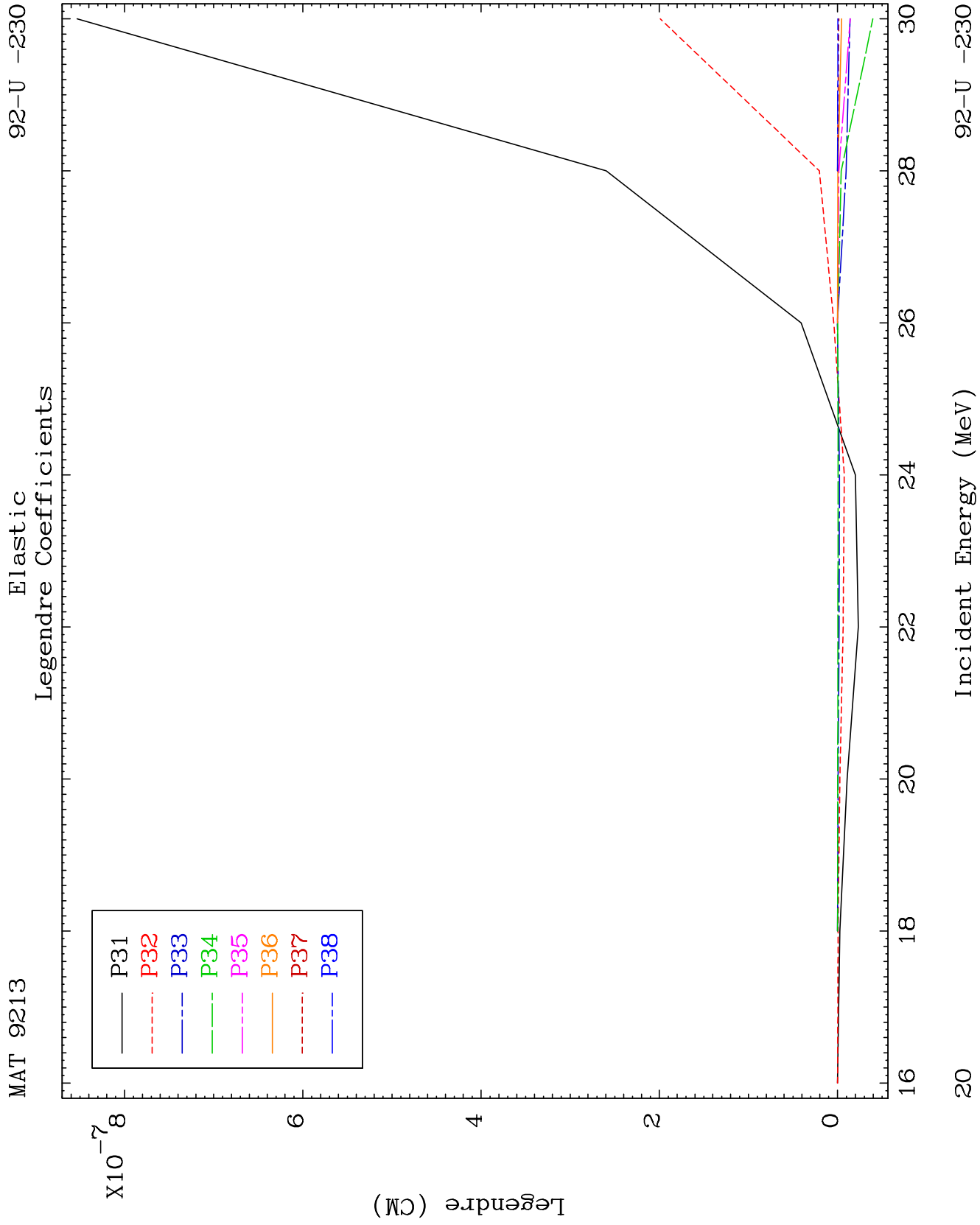
Incident Energy (MeV)

92-U -230





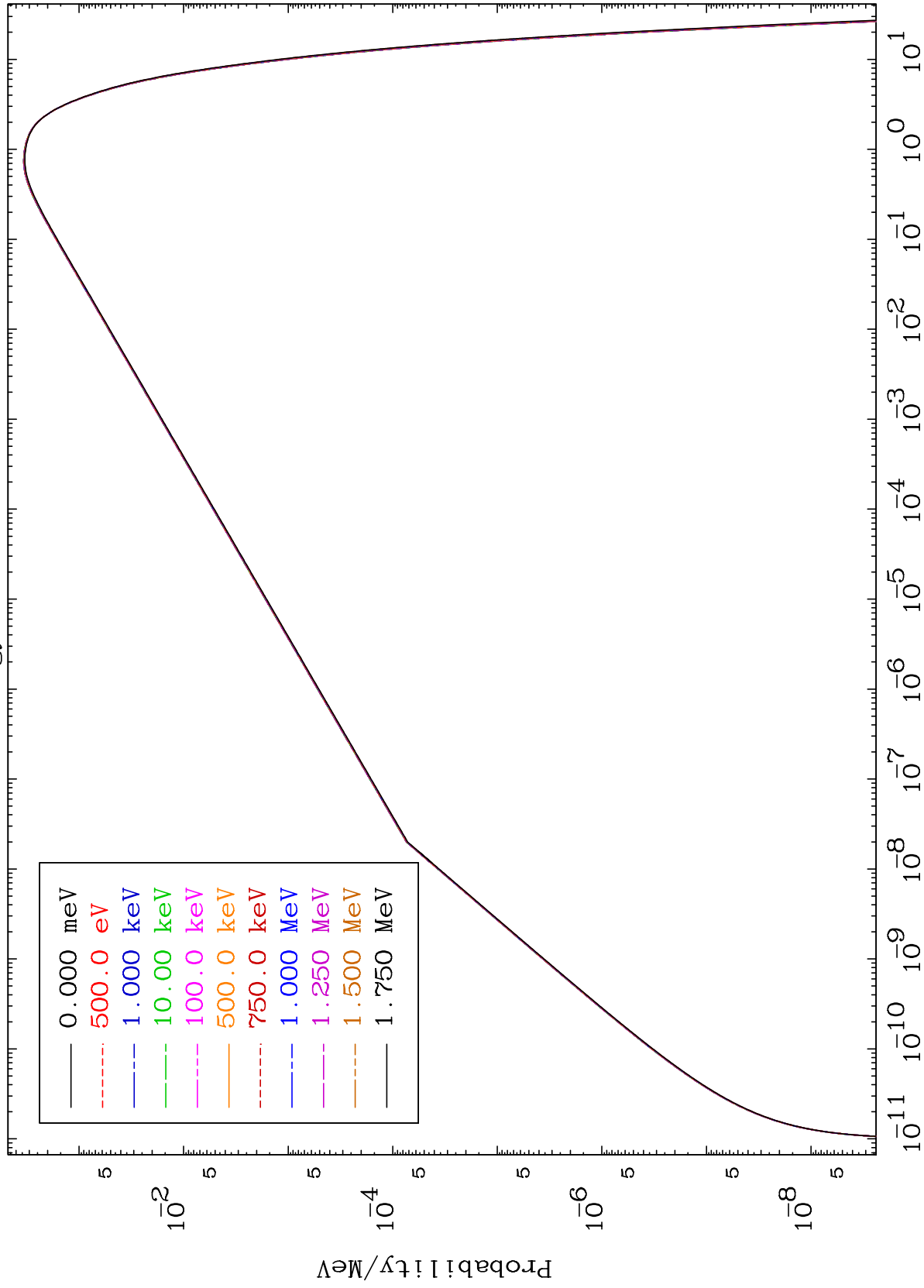




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Fission
Energy Distributions

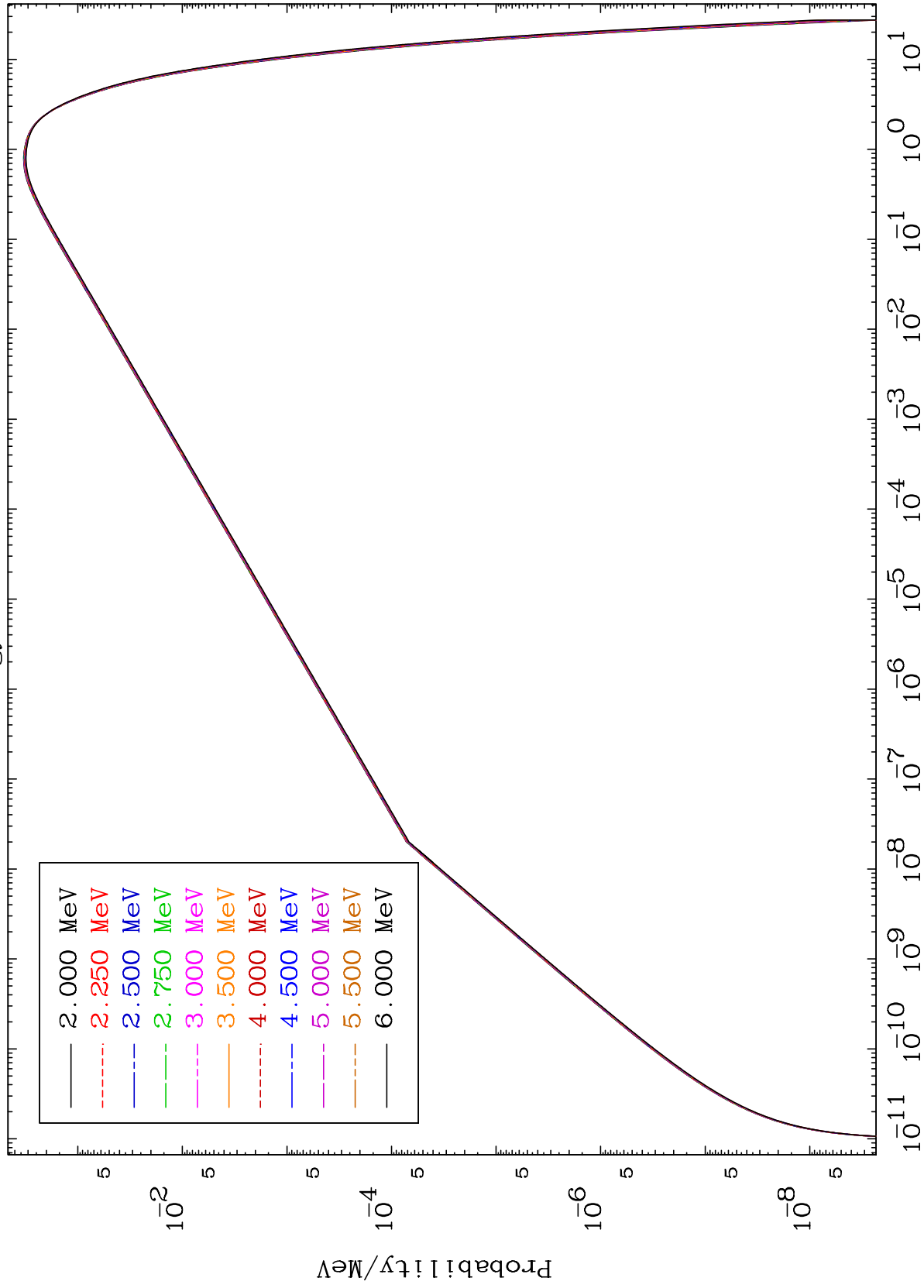
92-U -230



MAT 9213

Fission
Energy Distributions

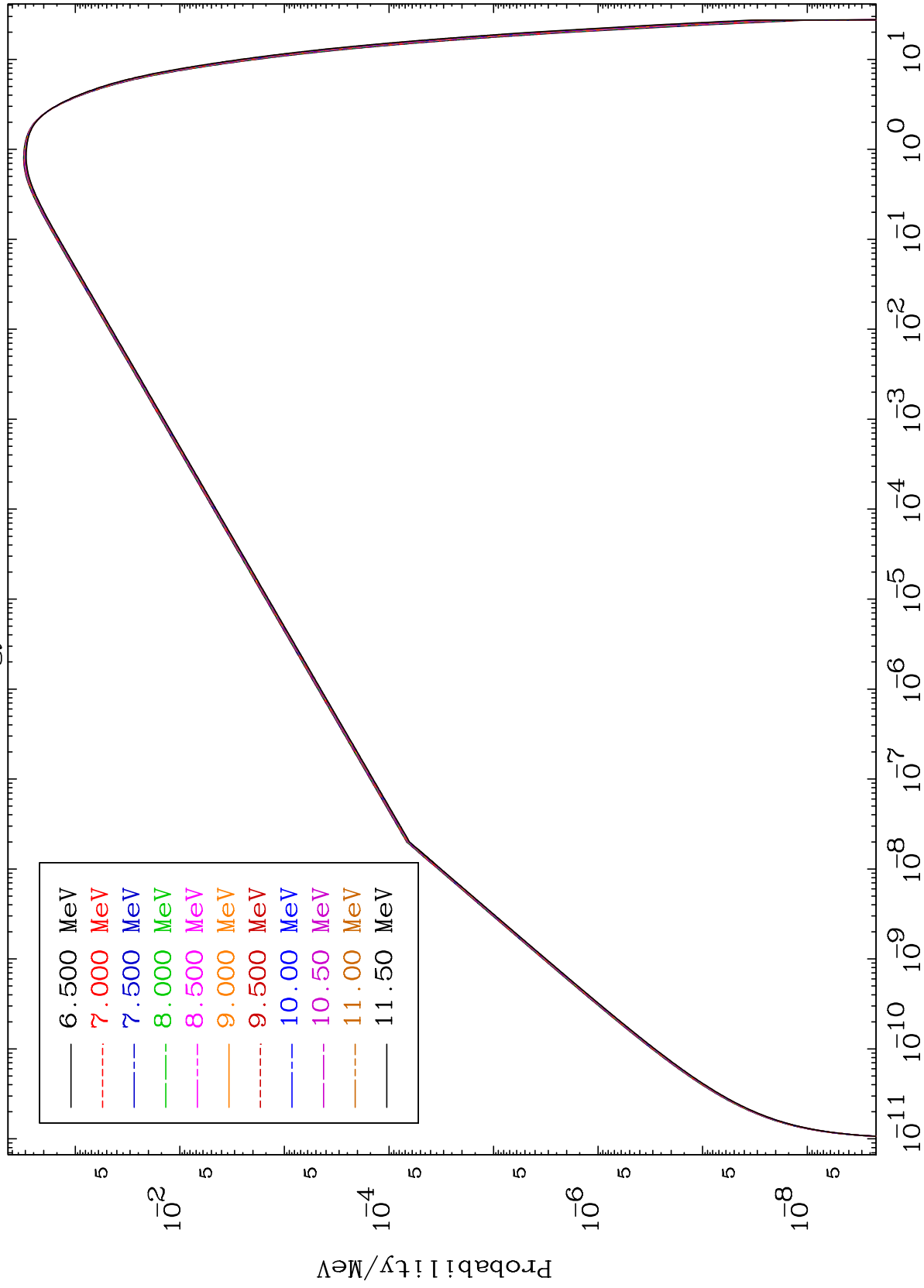
92-U -230



MAT 9213

Fission
Energy Distributions

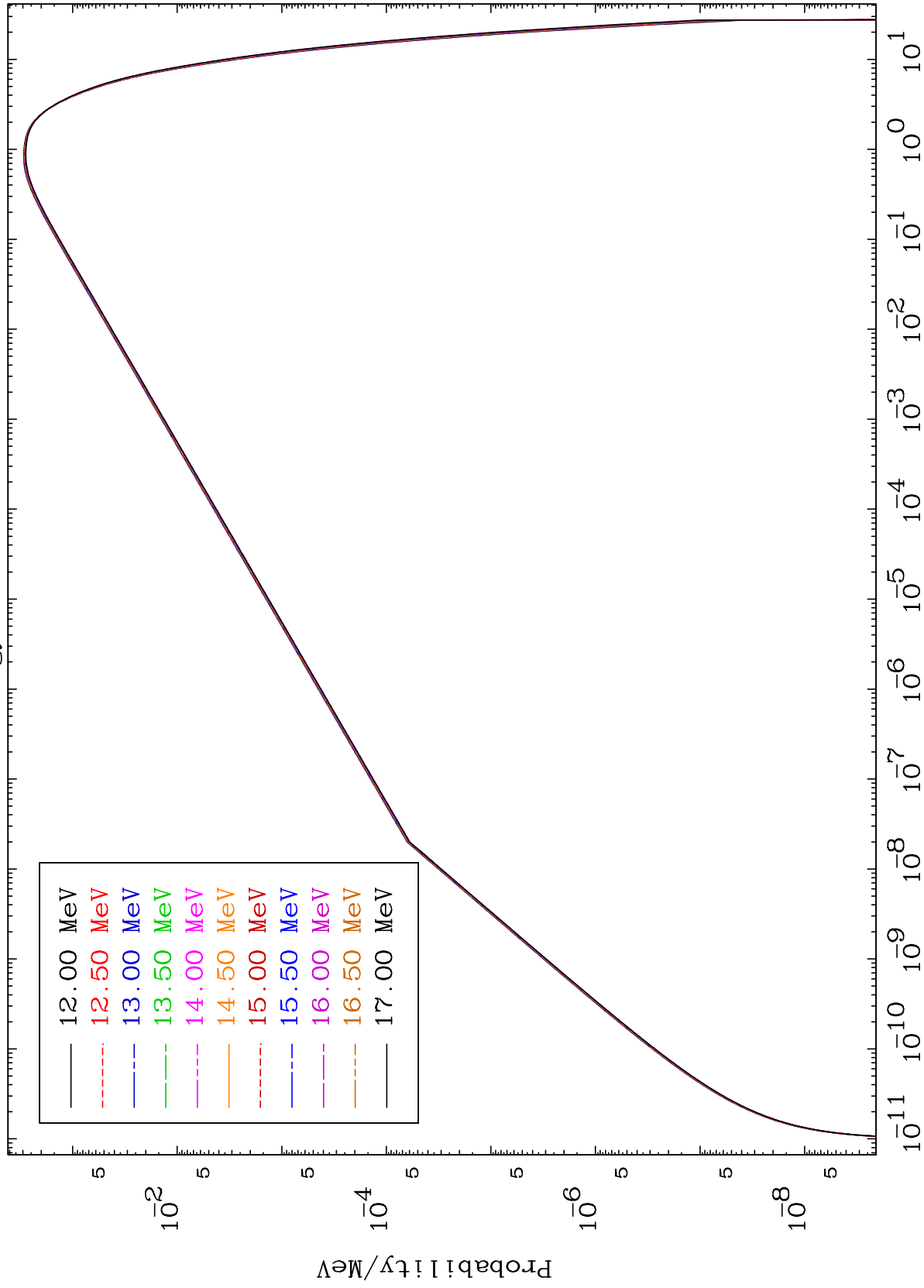
92-U -230



MAT 9213

Fission
Energy Distributions

92-U -230



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92-U -230

MAT 9213

Fission
Energy Distributions

92-U -230

