

Program EVALPLOT  
(Version 2021-1)

by

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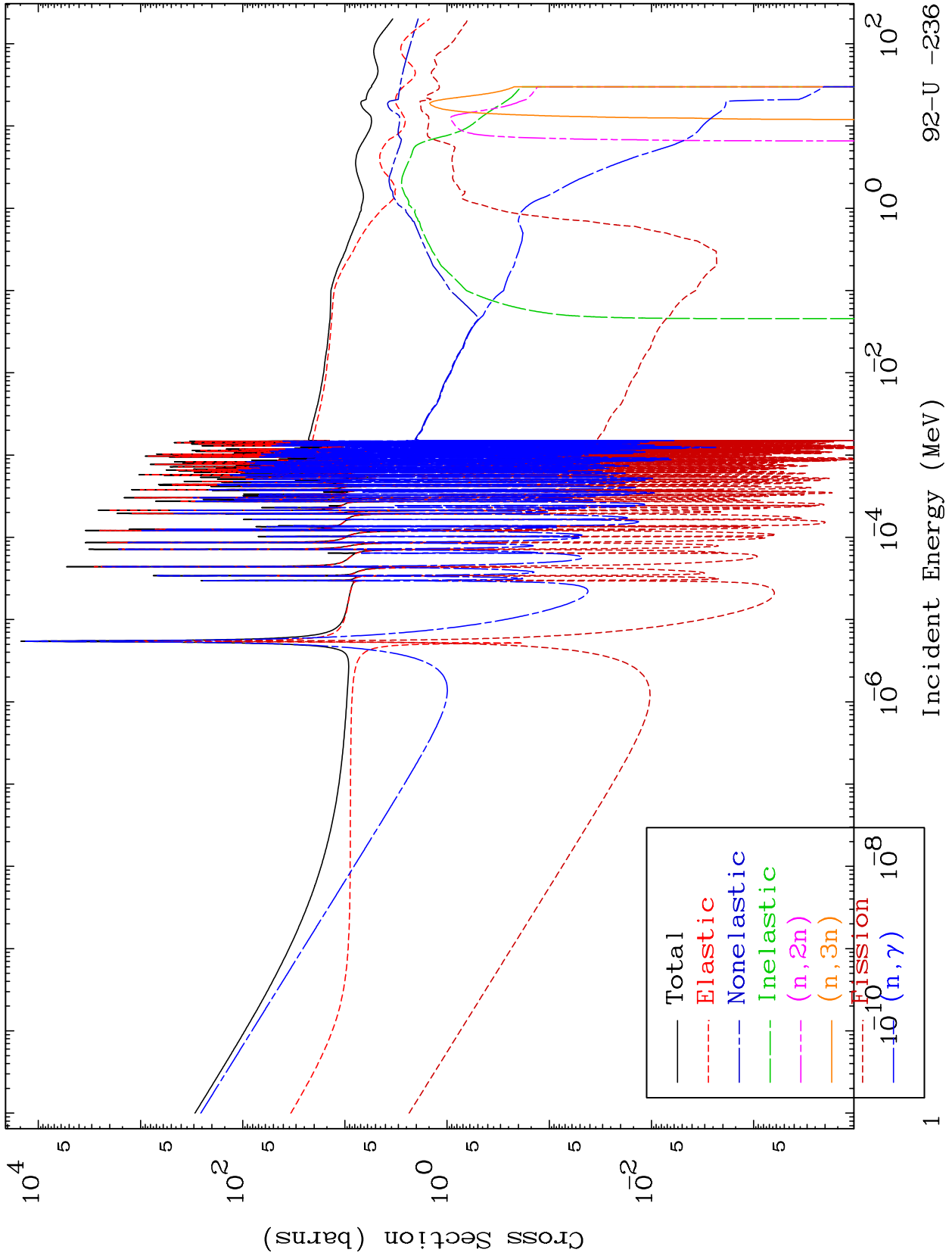
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Press Mouse Button to Start

MAT 9231

Neutron Major  
293 Kelvin Cross Sections

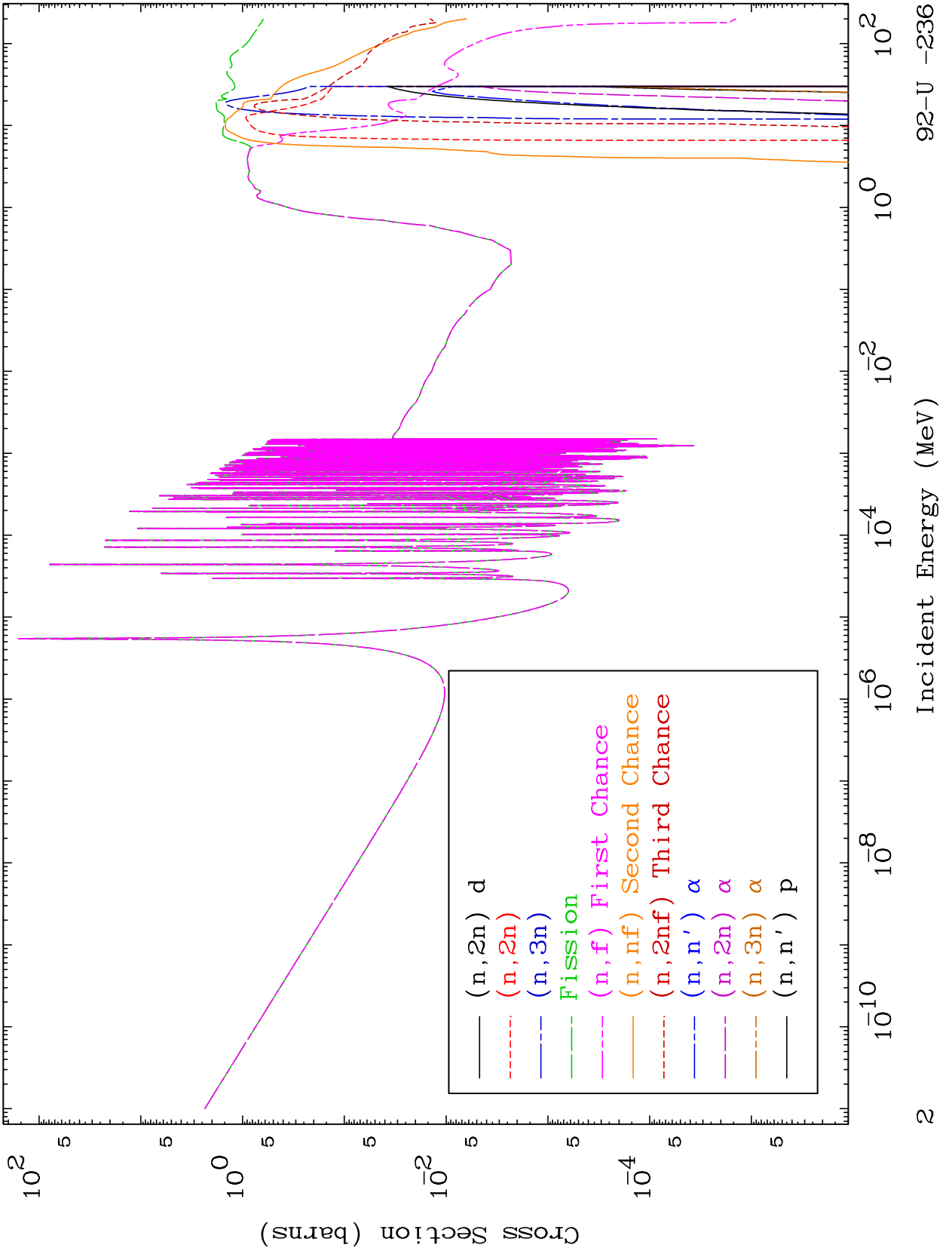
92-U -236

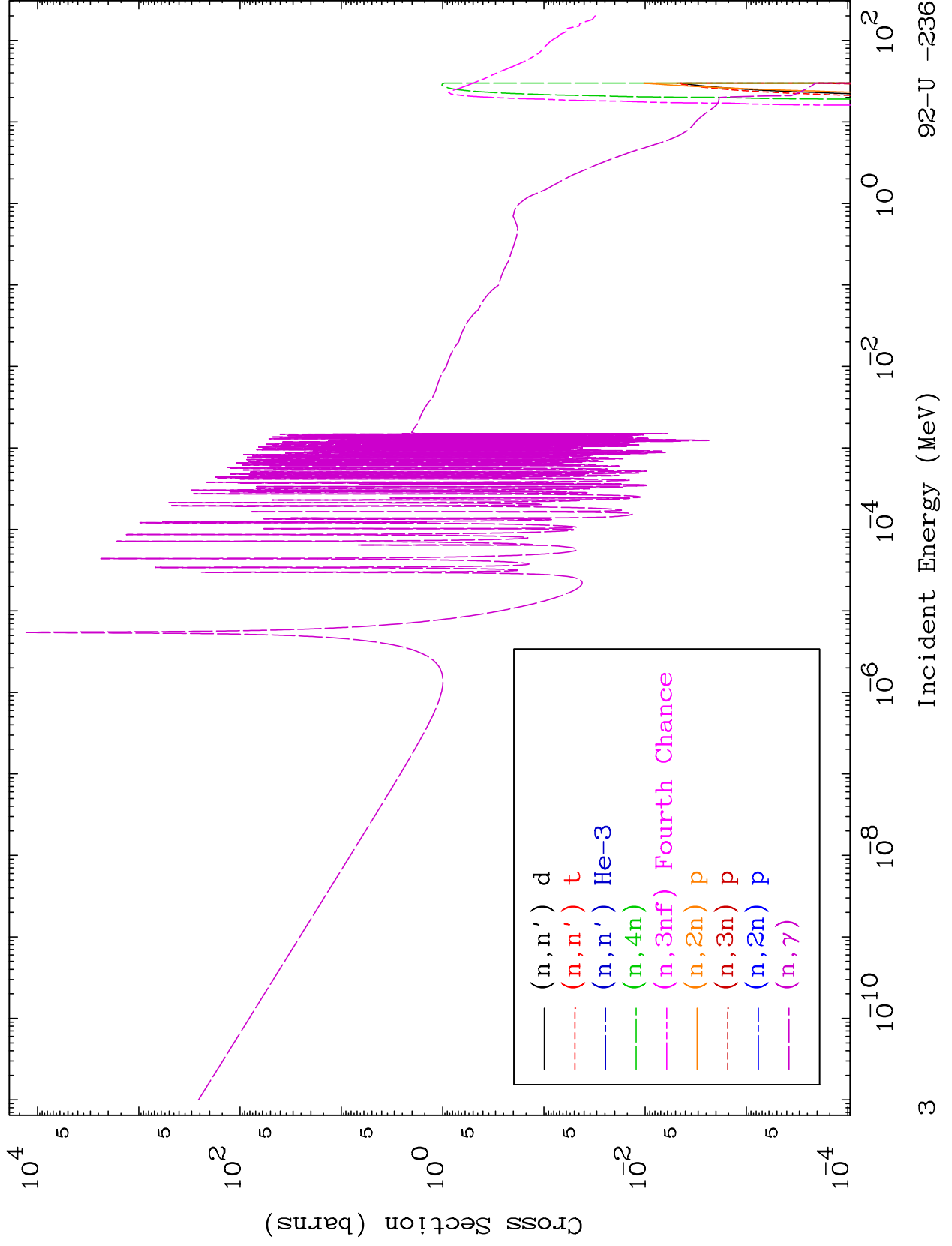


MAT 9231

Neutron Absorption  
293 Kelvin Cross Sections

92-U -236

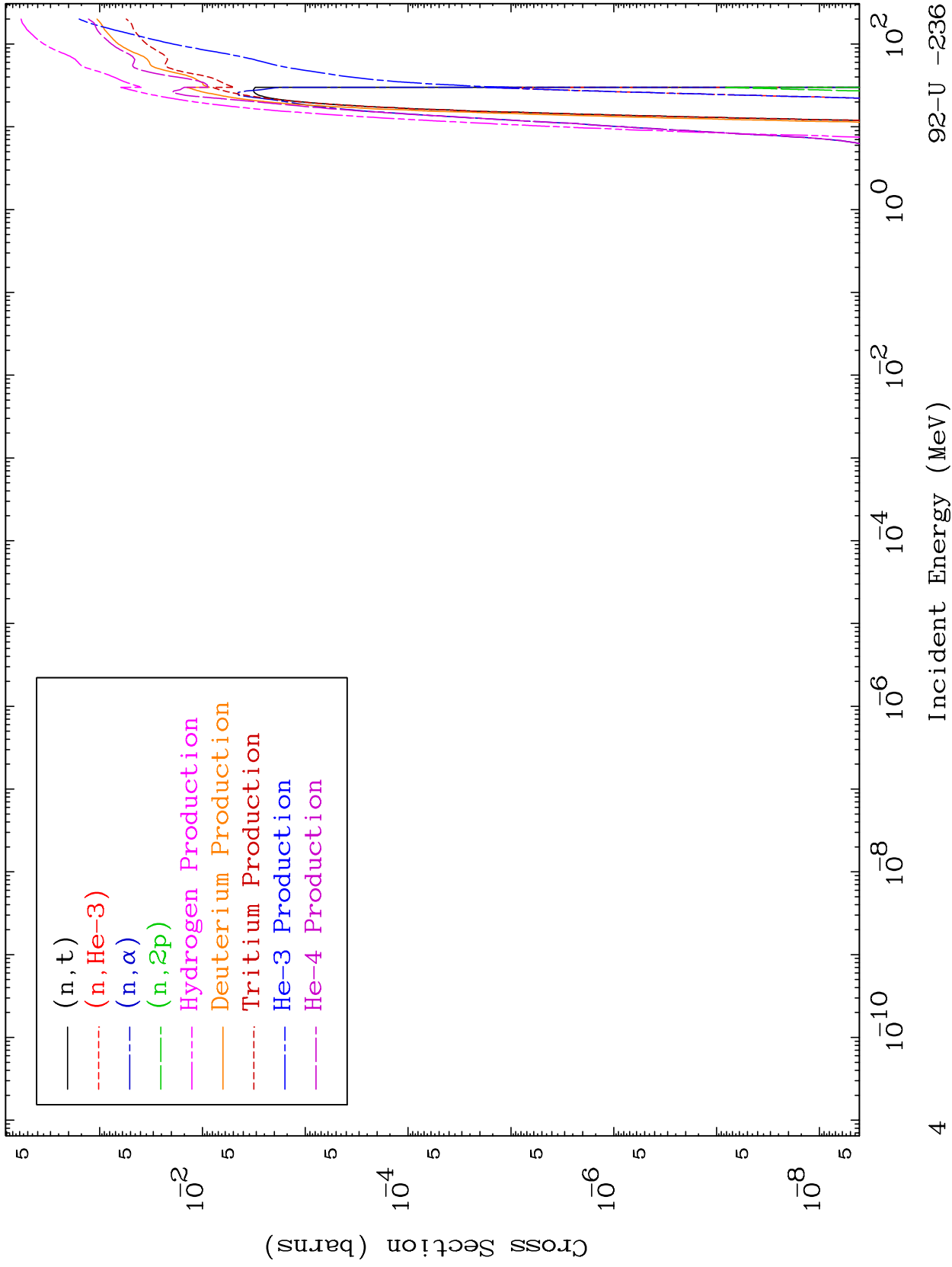




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Neutron Absorption  
293 Kelvin Cross Sections

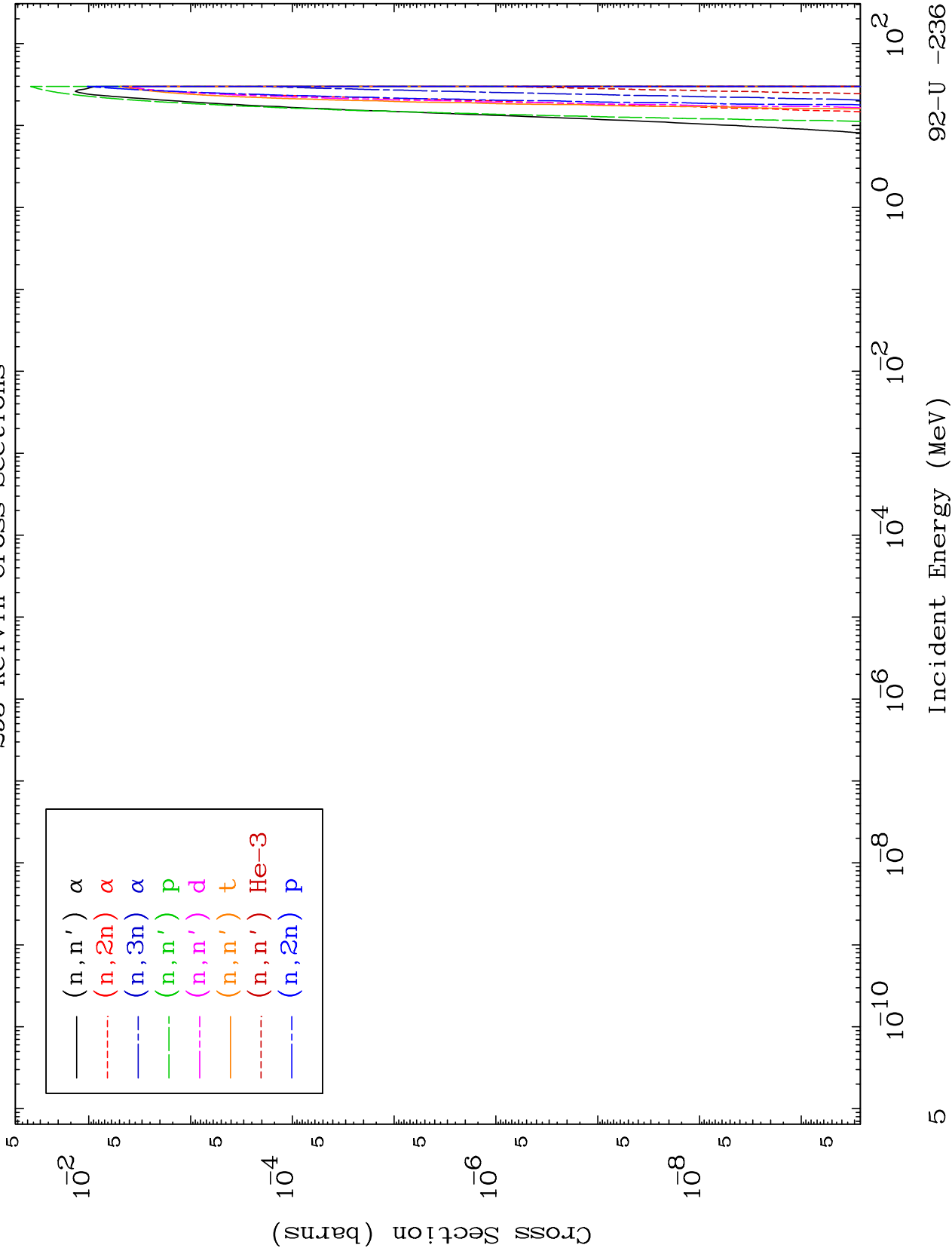
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Charged Particle  
293 Kelvin Cross Sections

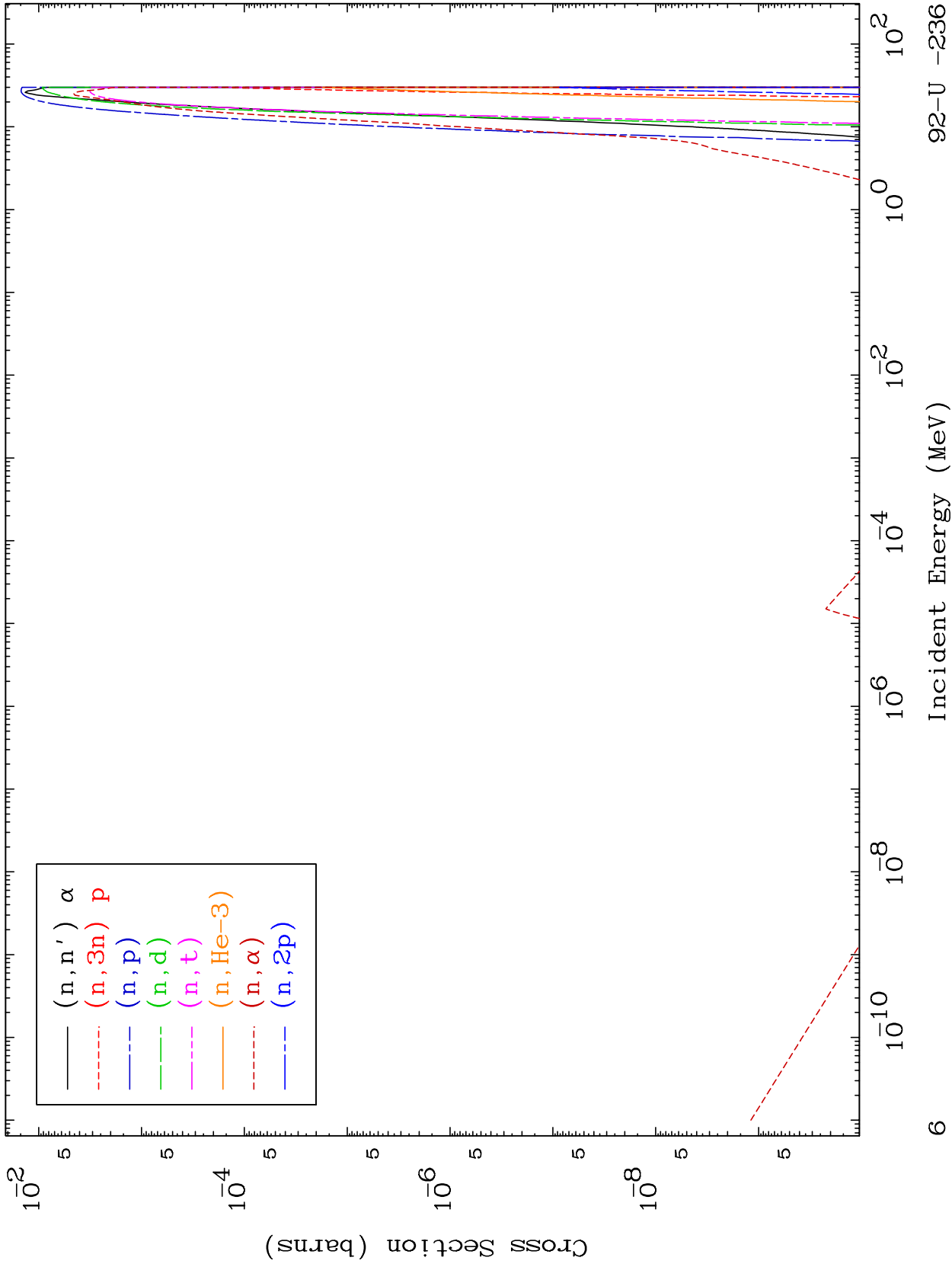
92-U -236



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Charged Particle  
293 Kelvin Cross Sections

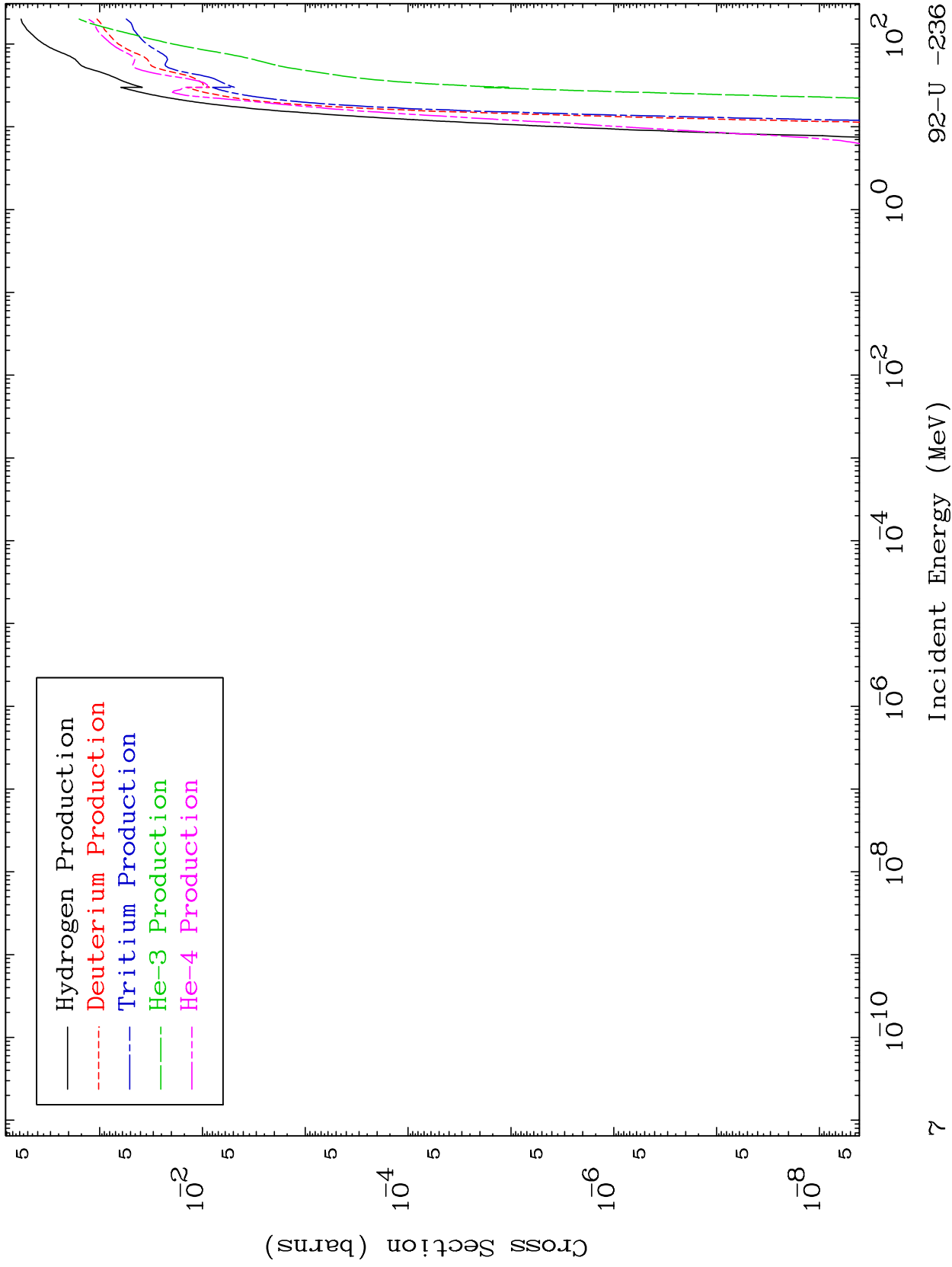
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Particle Production  
293 Kelvin Cross Sections

92-U -236

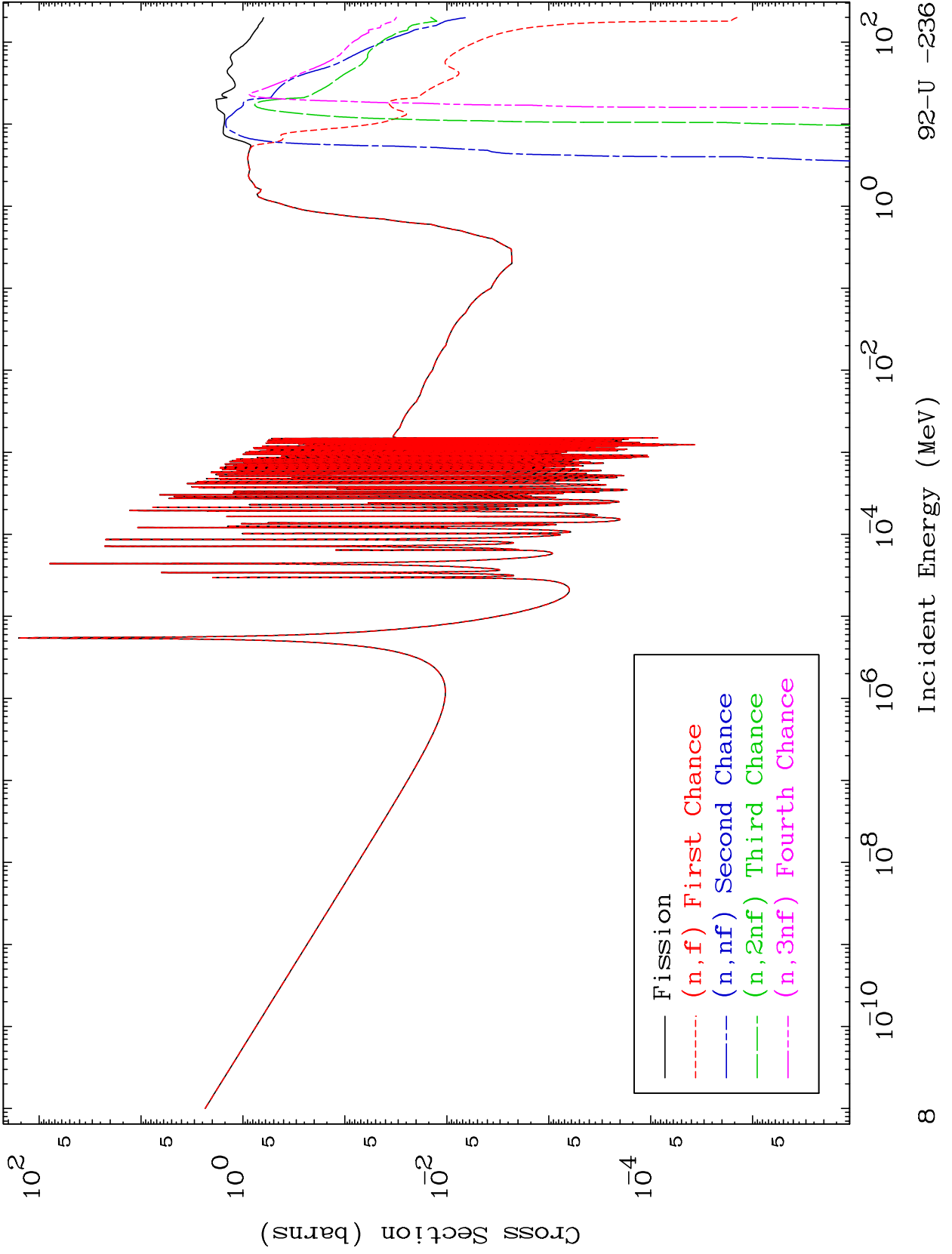


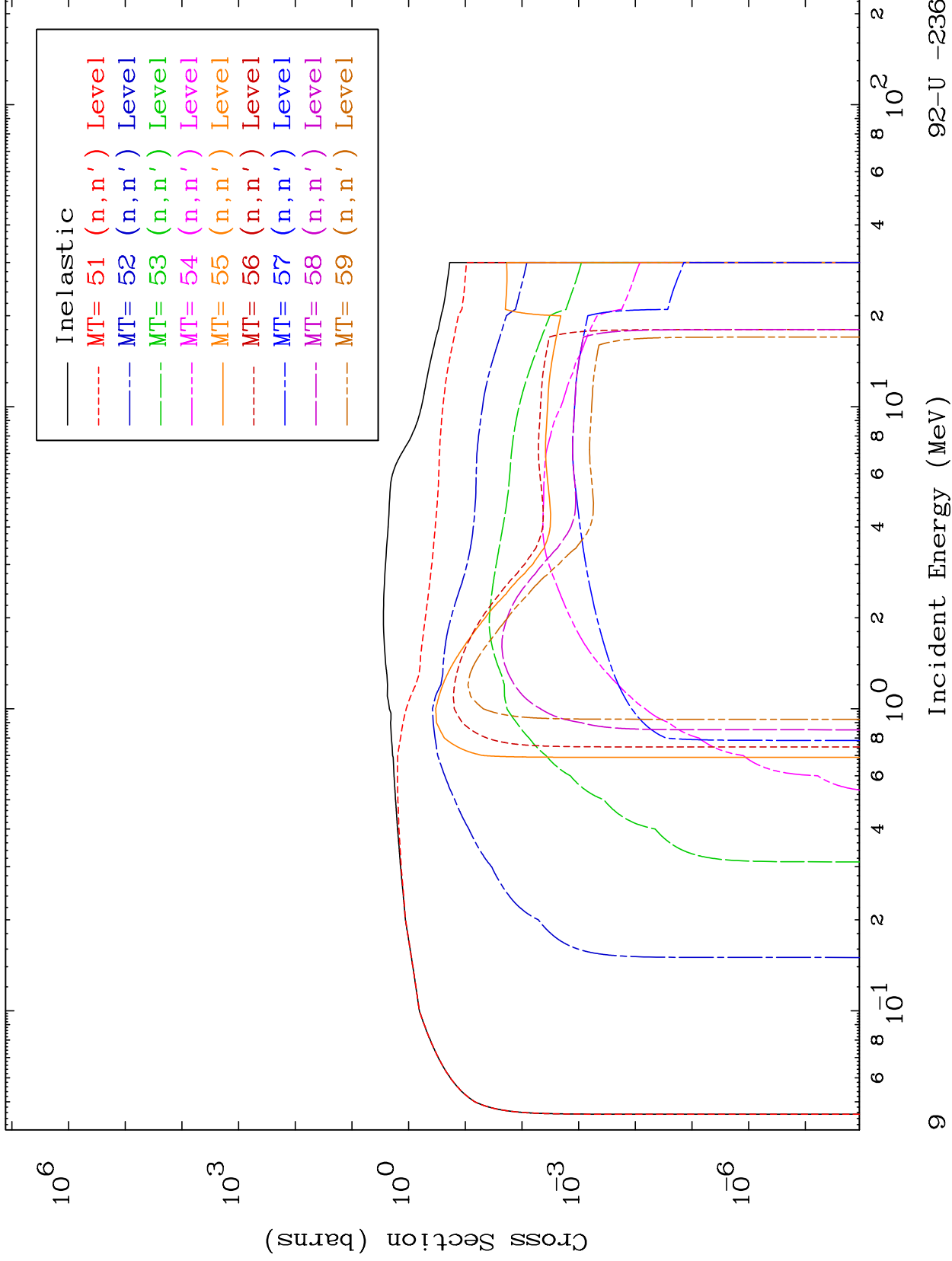


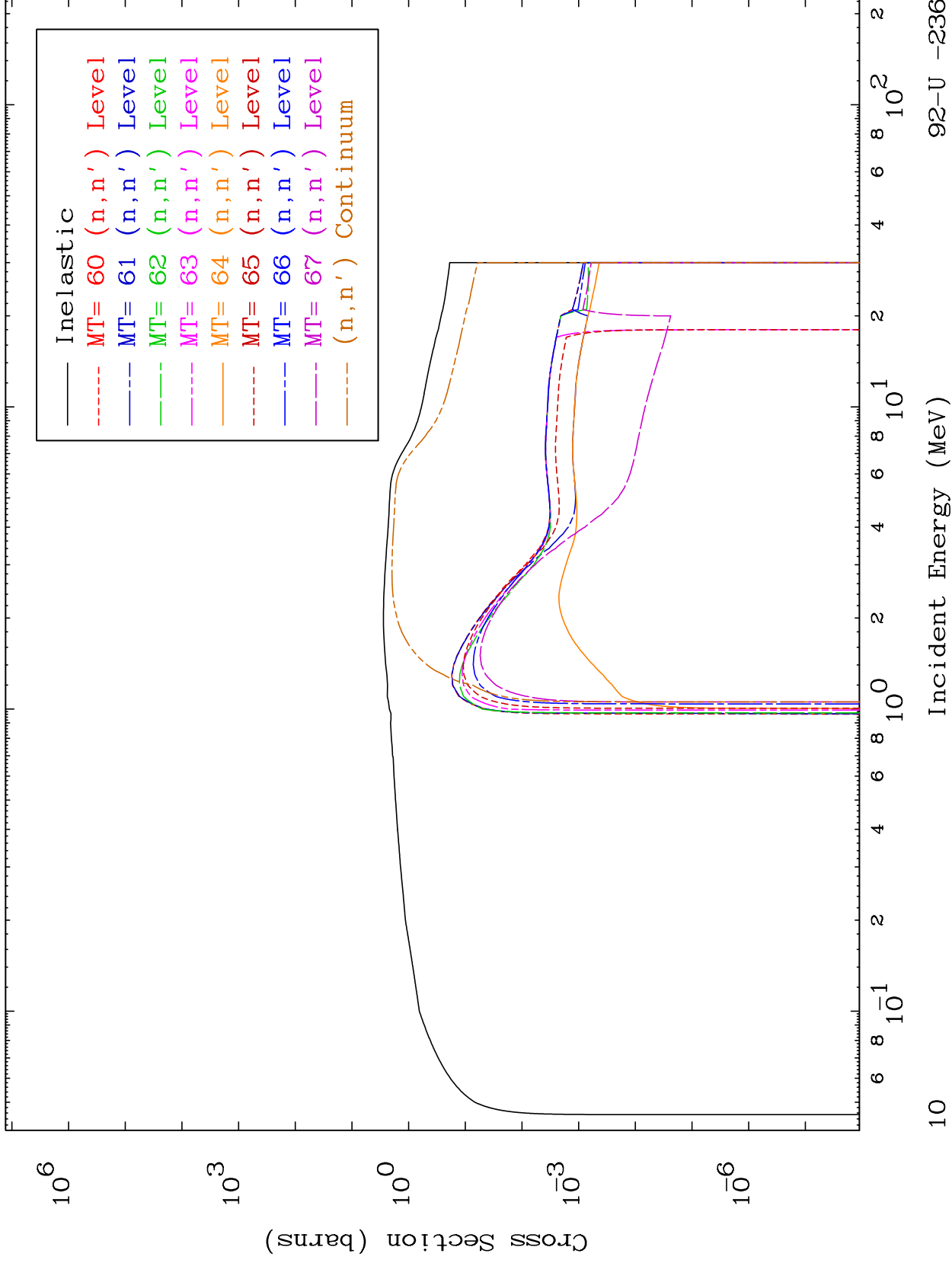
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Fission  
293 Kelvin Cross Sections

92-U -236



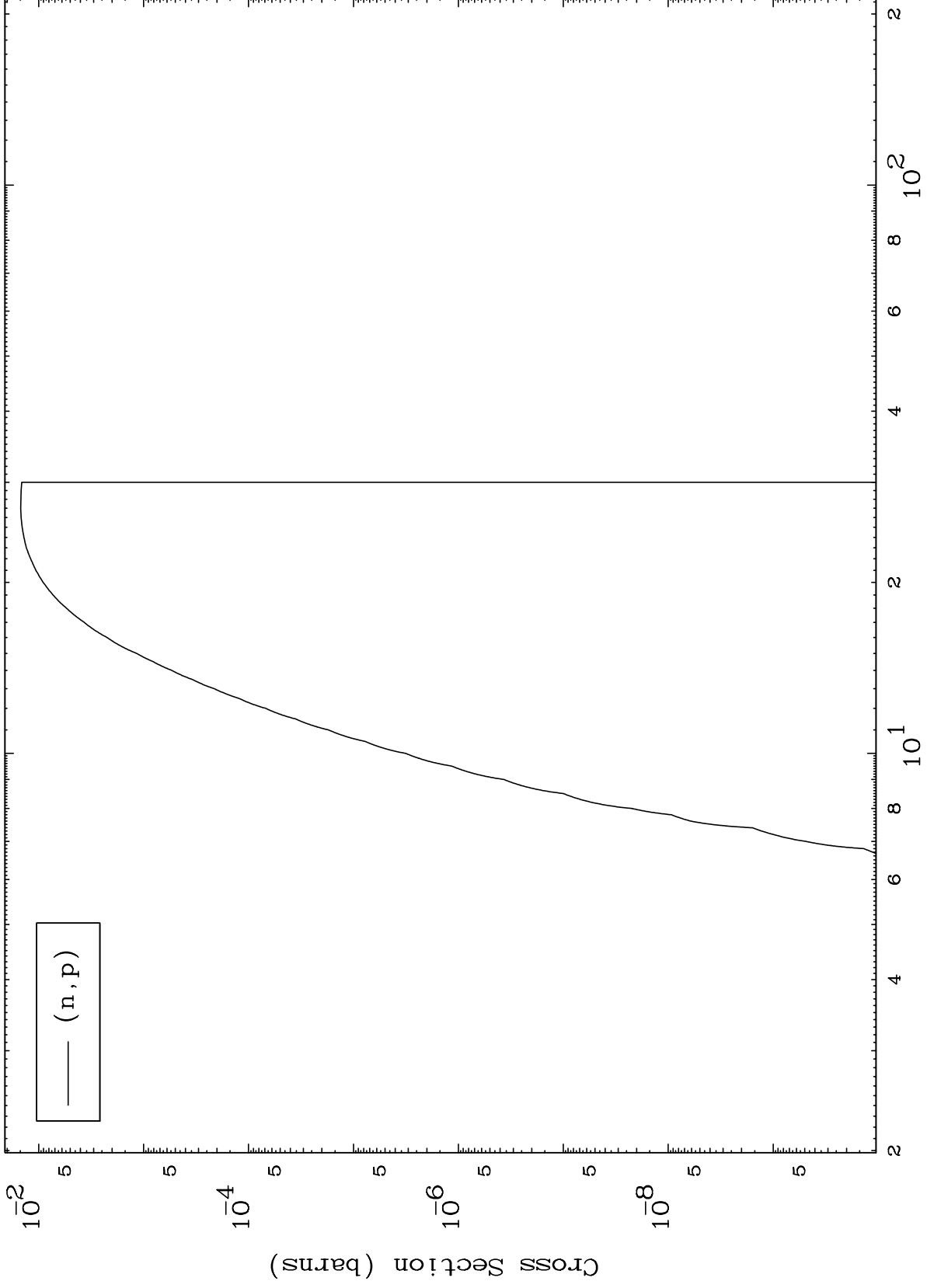




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(n,p) Levels  
293 Kelvin Cross Sections

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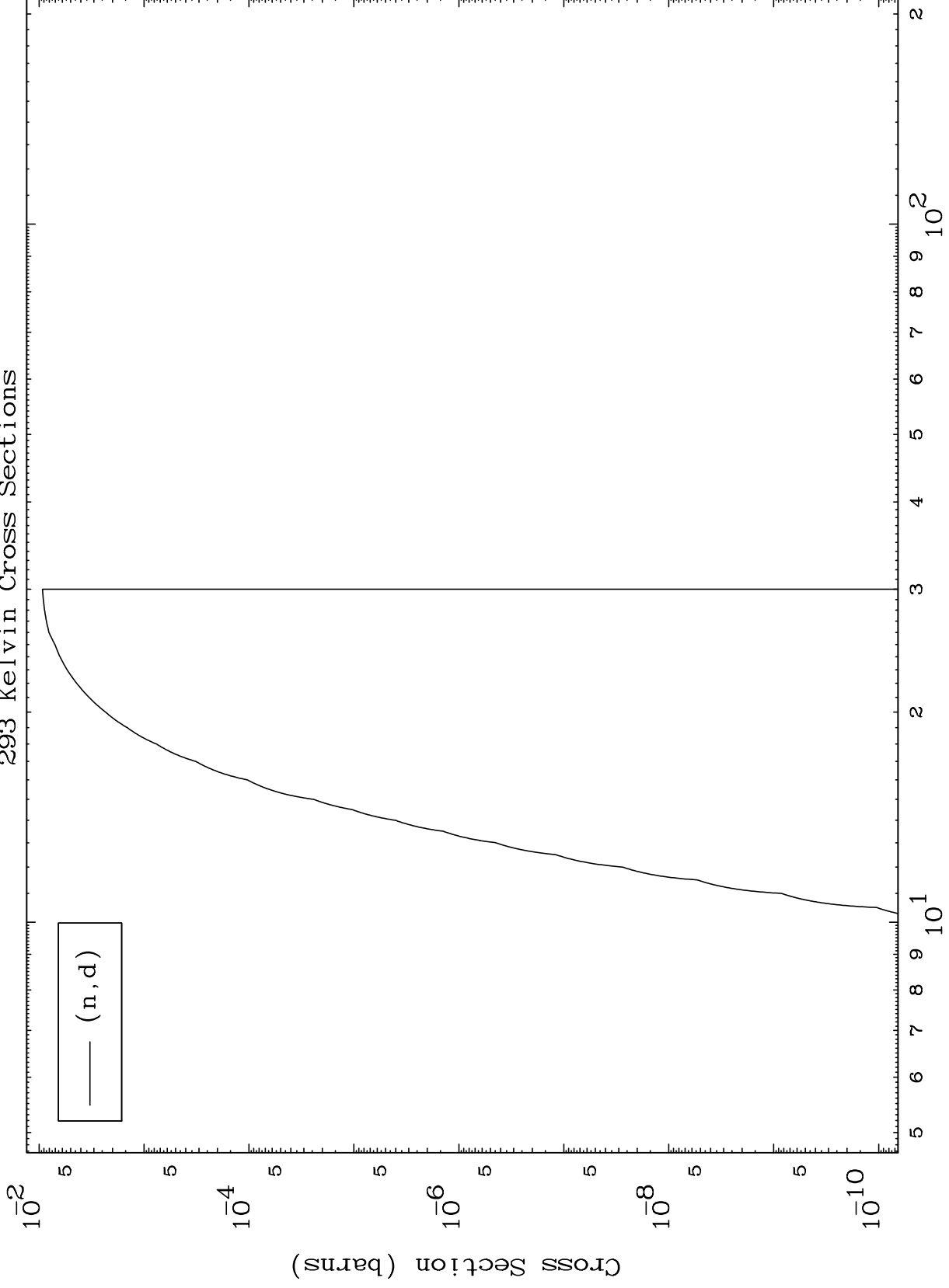
Incident Energy (MeV)

92-U -236

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(n,d) Levels  
293 Kelvin Cross Sections

92-U -236



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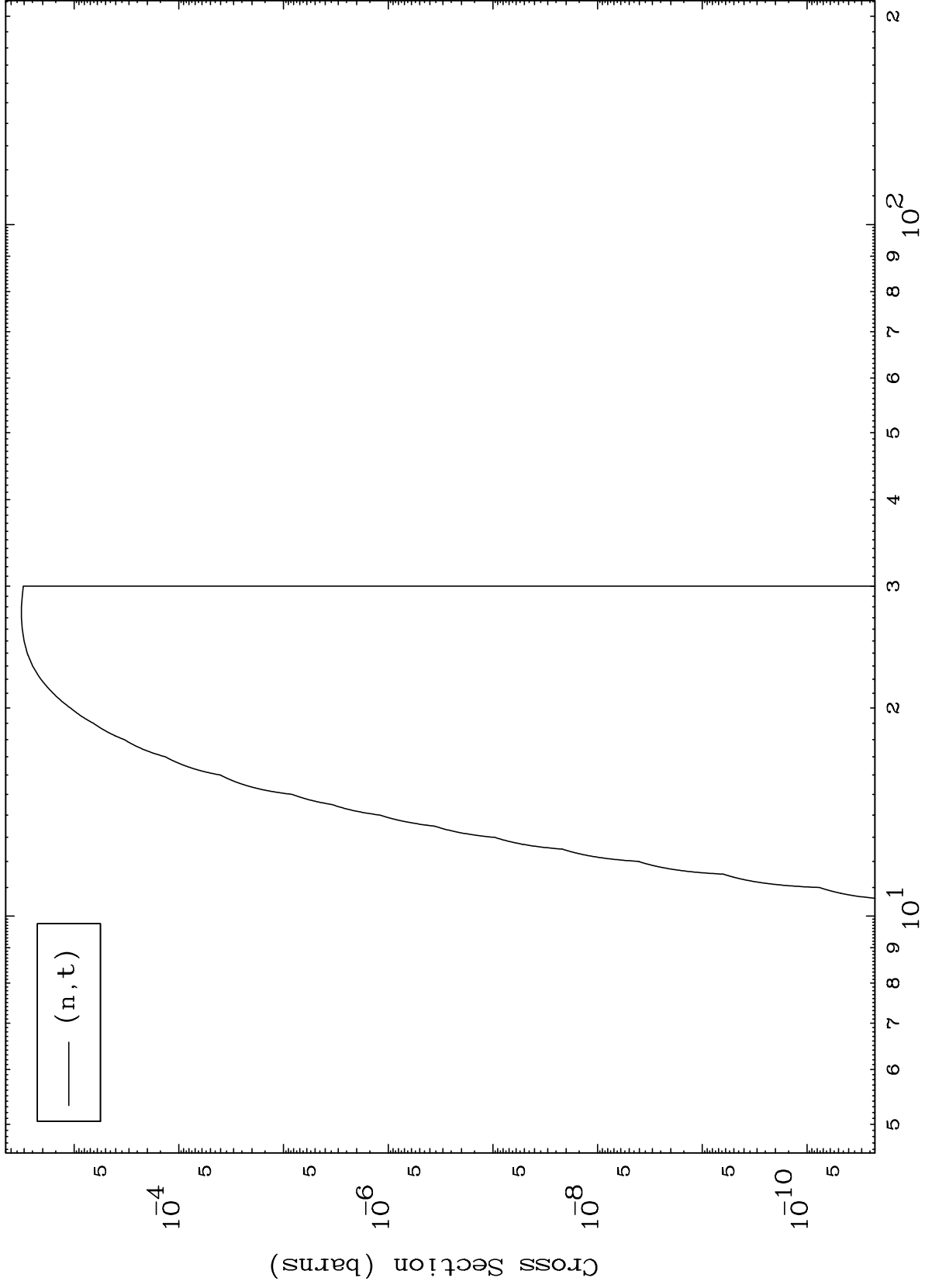
Incident Energy (MeV)

92-U -236

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(n,t) Levels  
293 Kelvin Cross Sections

92-U -236



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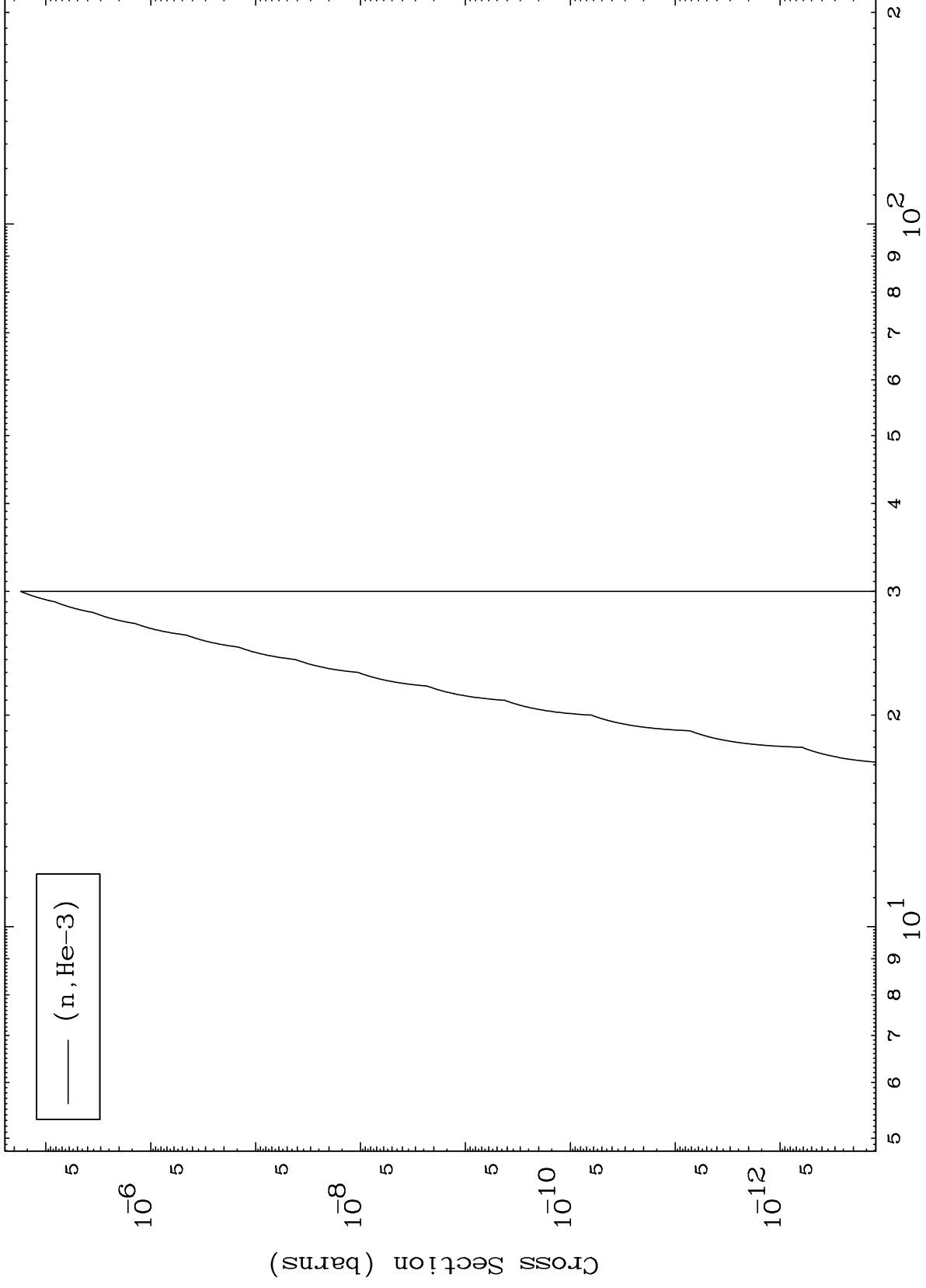
Incident Energy (MeV)

92-U -236

MAT 9231

(n,He3) Levels  
293 Kelvin Cross Sections

92-U -236



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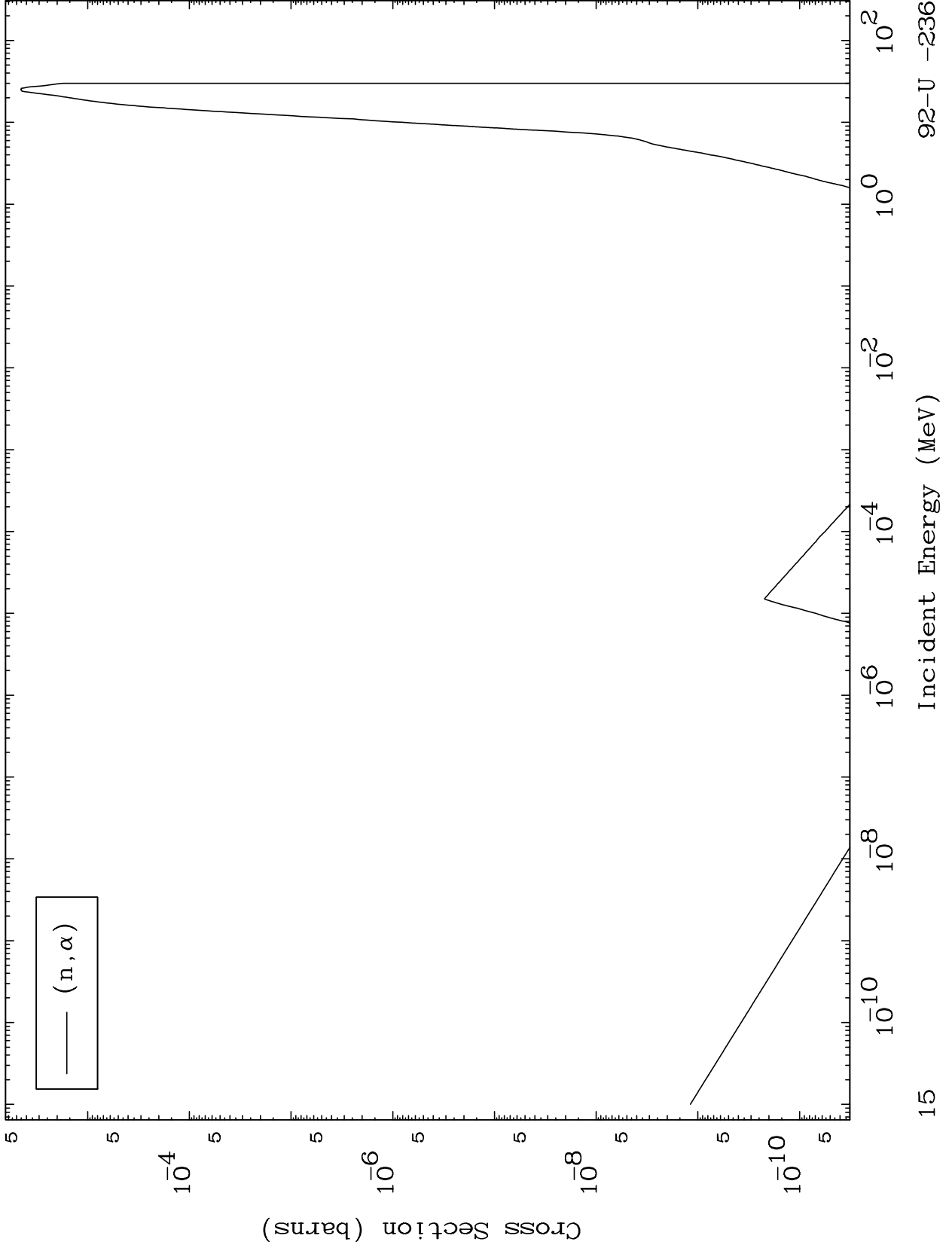
Incident Energy (MeV)

92-U -236

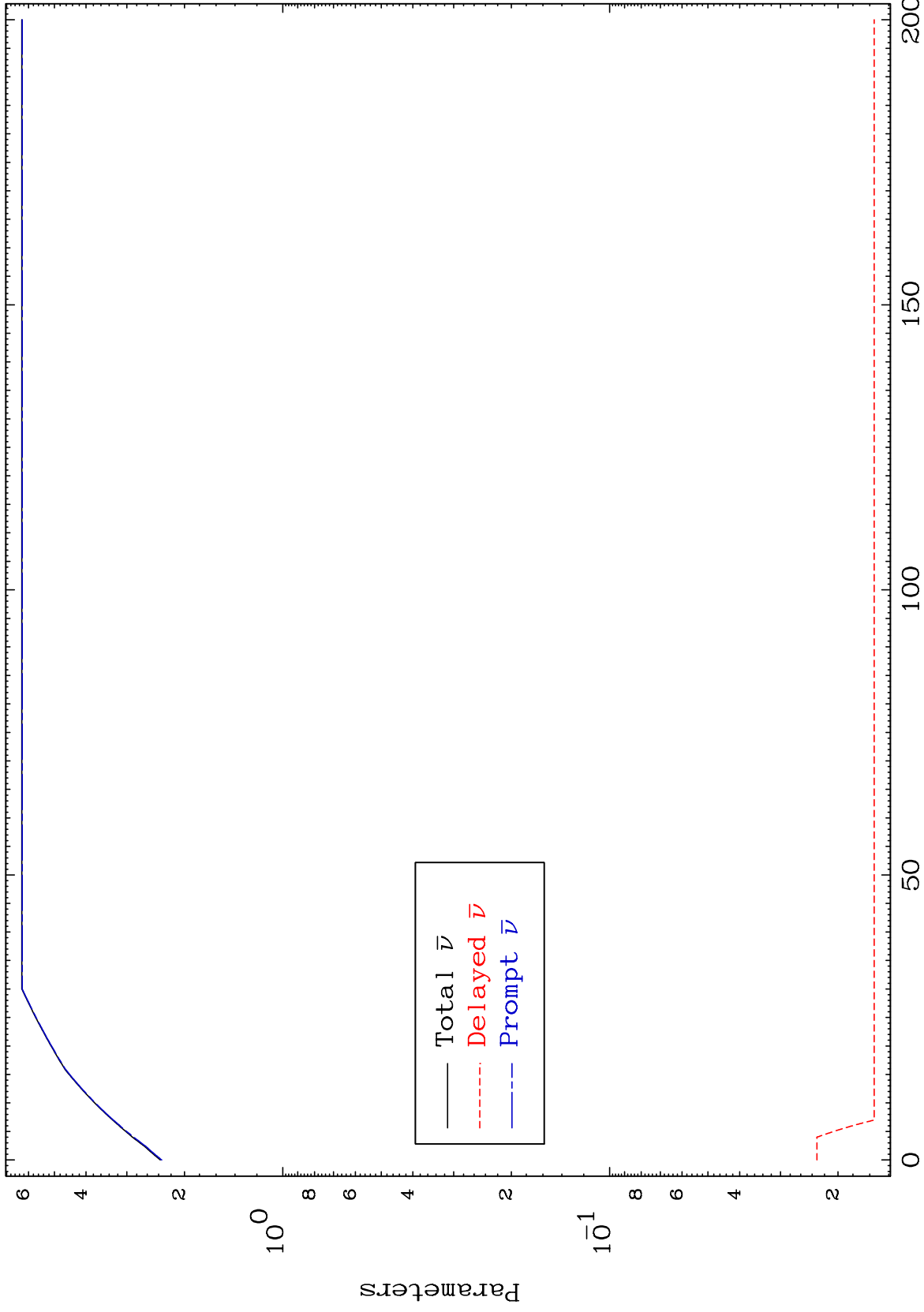
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(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

92-U -236



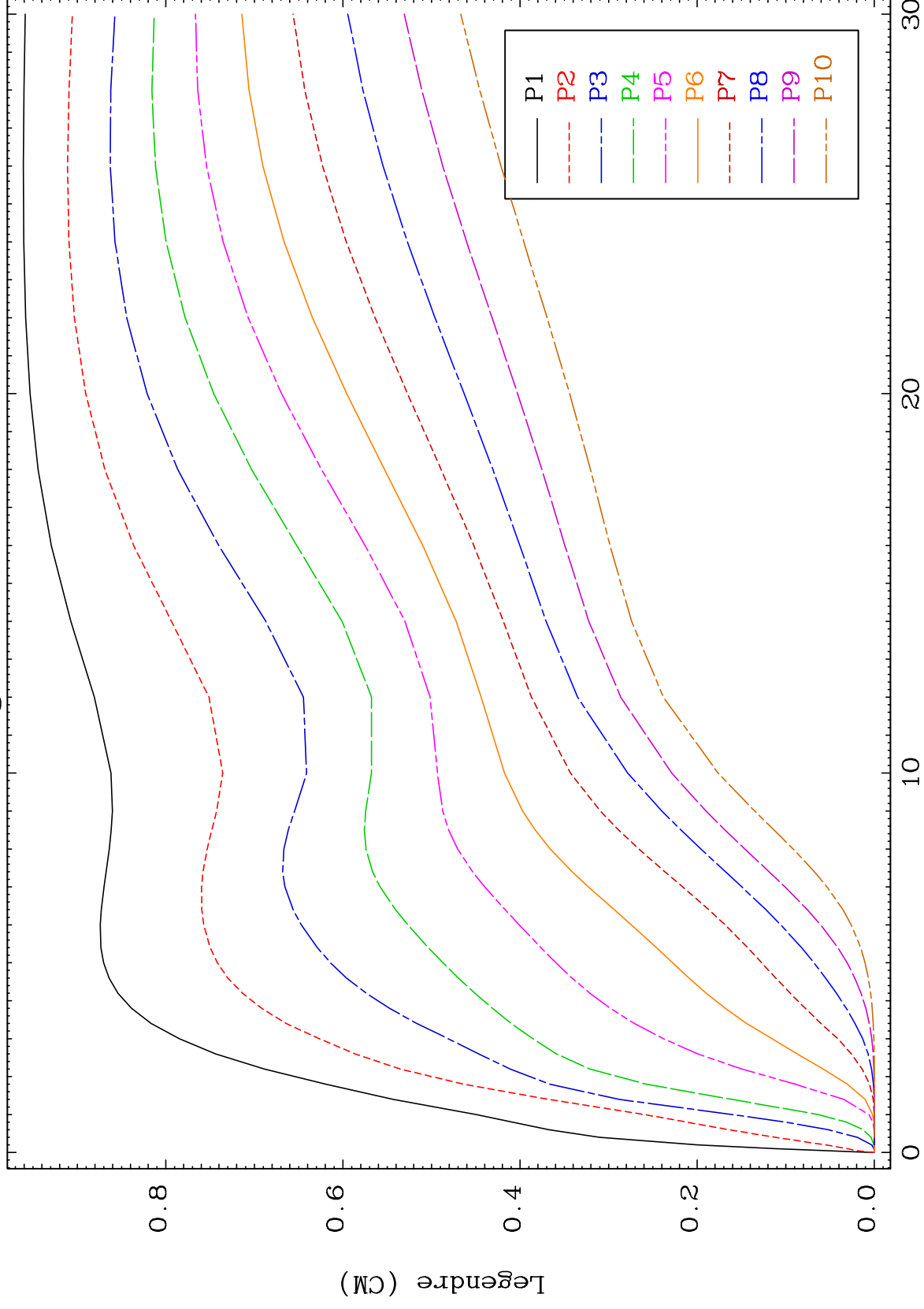




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Elastic Legendre Coefficients

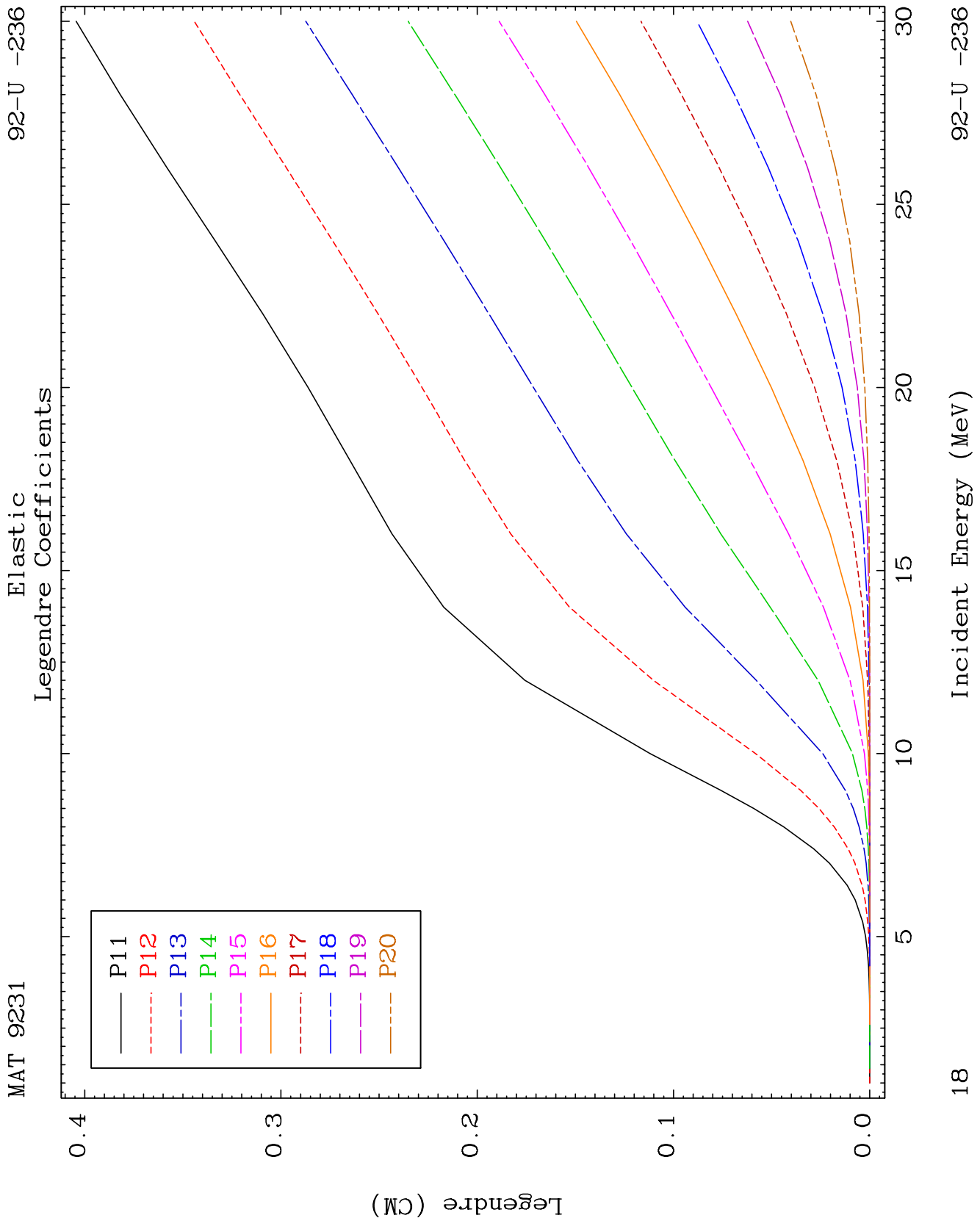
92-U -236



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Incident Energy (MeV)

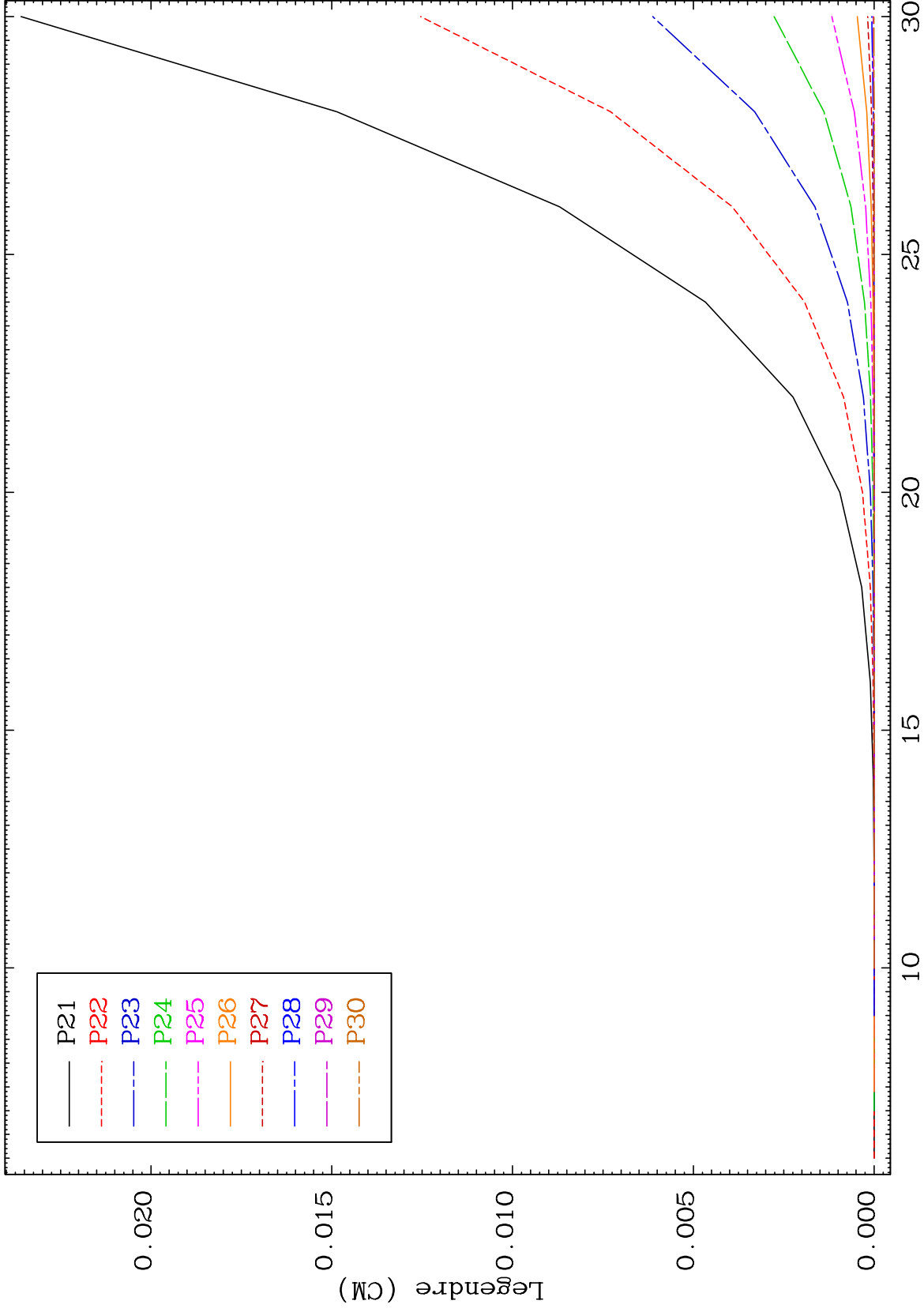
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### Elastic Legendre Coefficients

92-U -236



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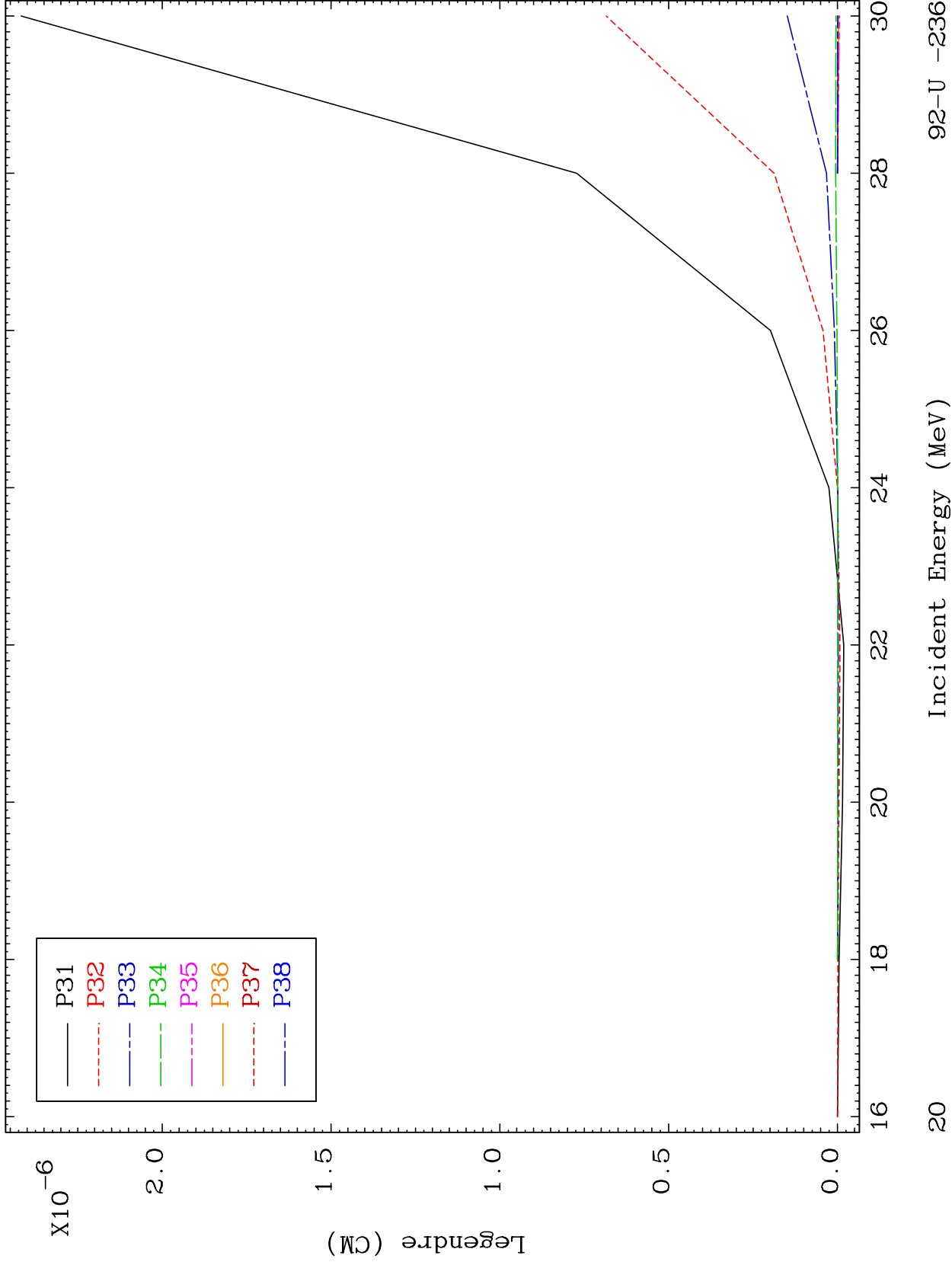
Incident Energy (MeV)

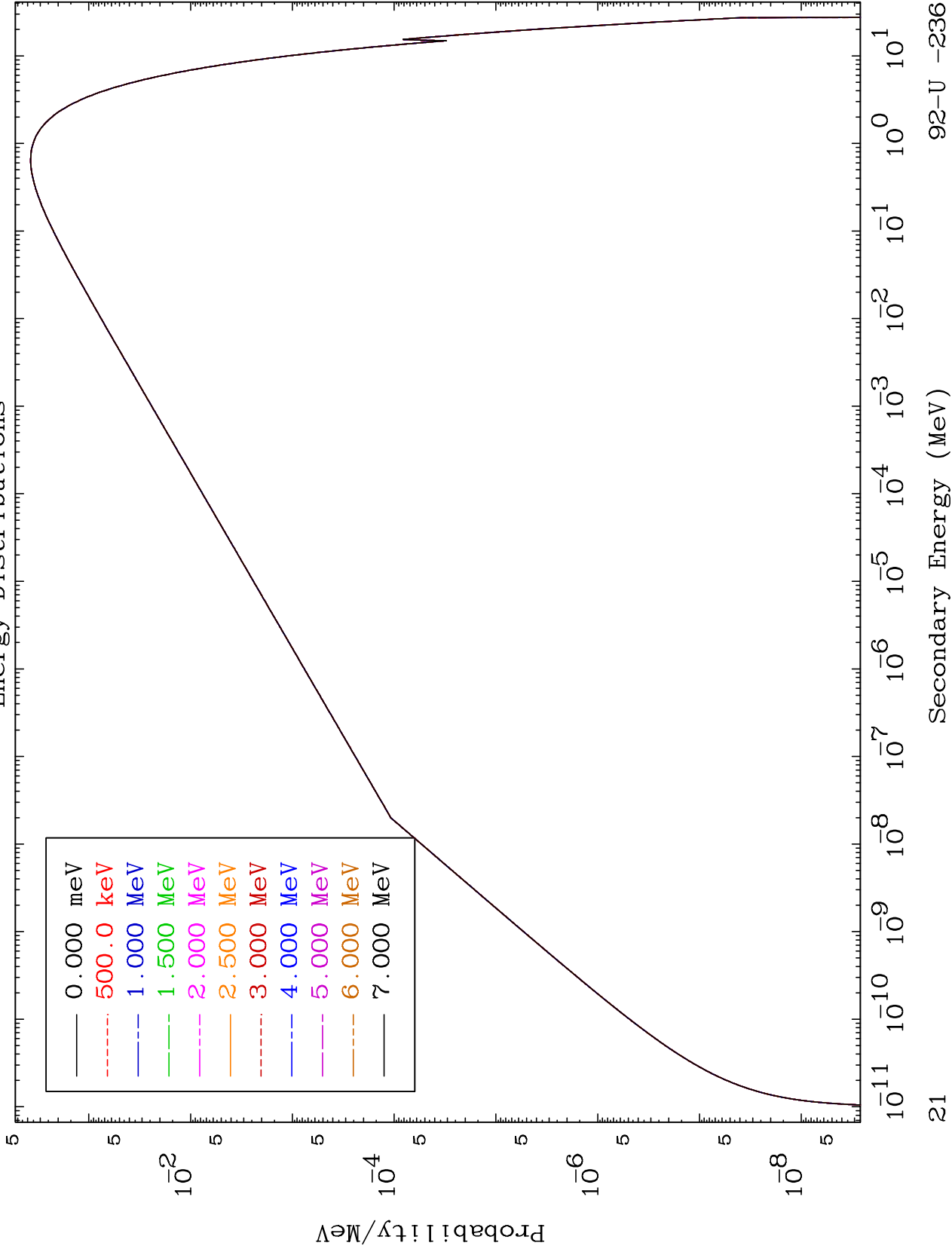
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Elastic  
Legendre Coefficients

92-U -236

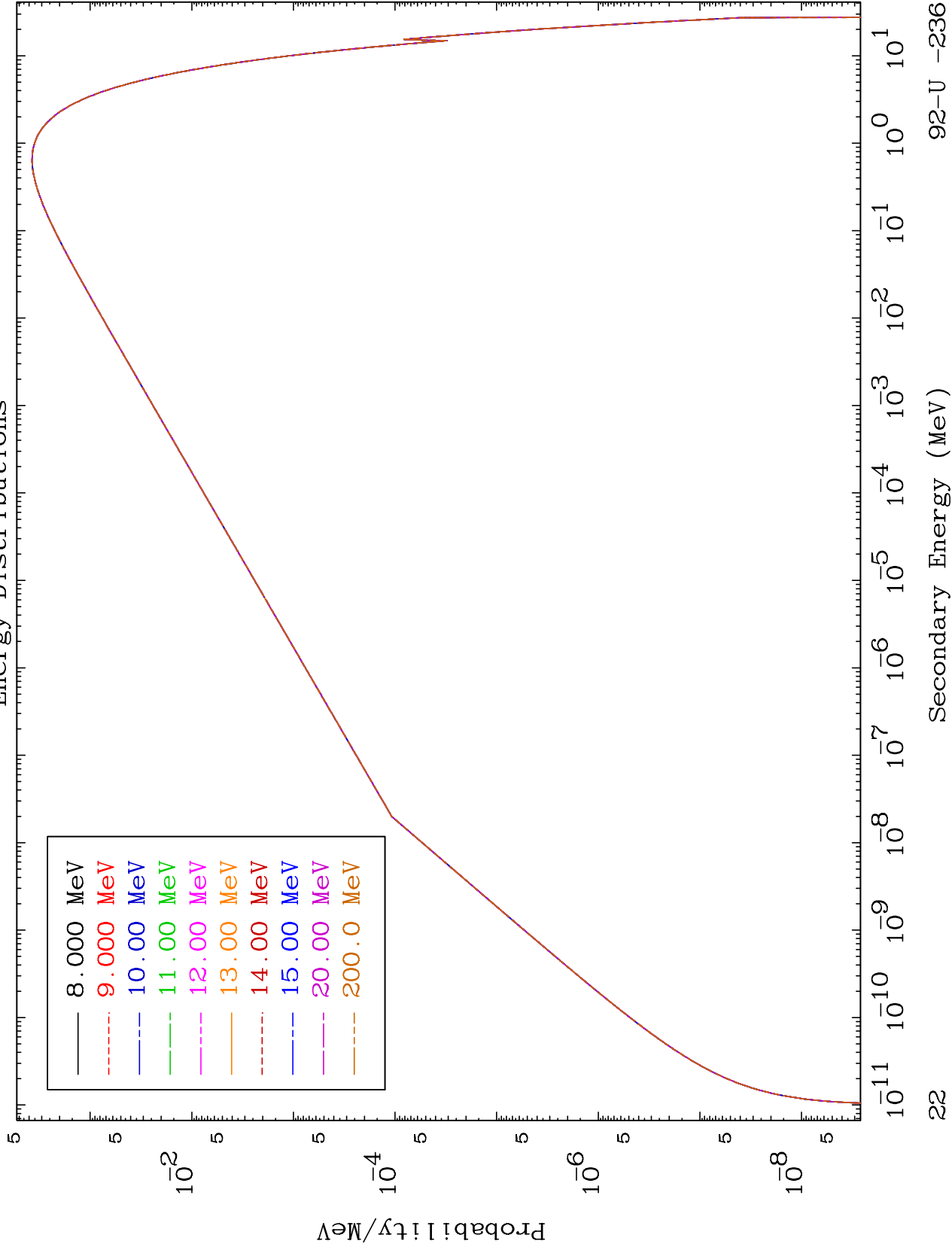




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### Fission Energy Distributions

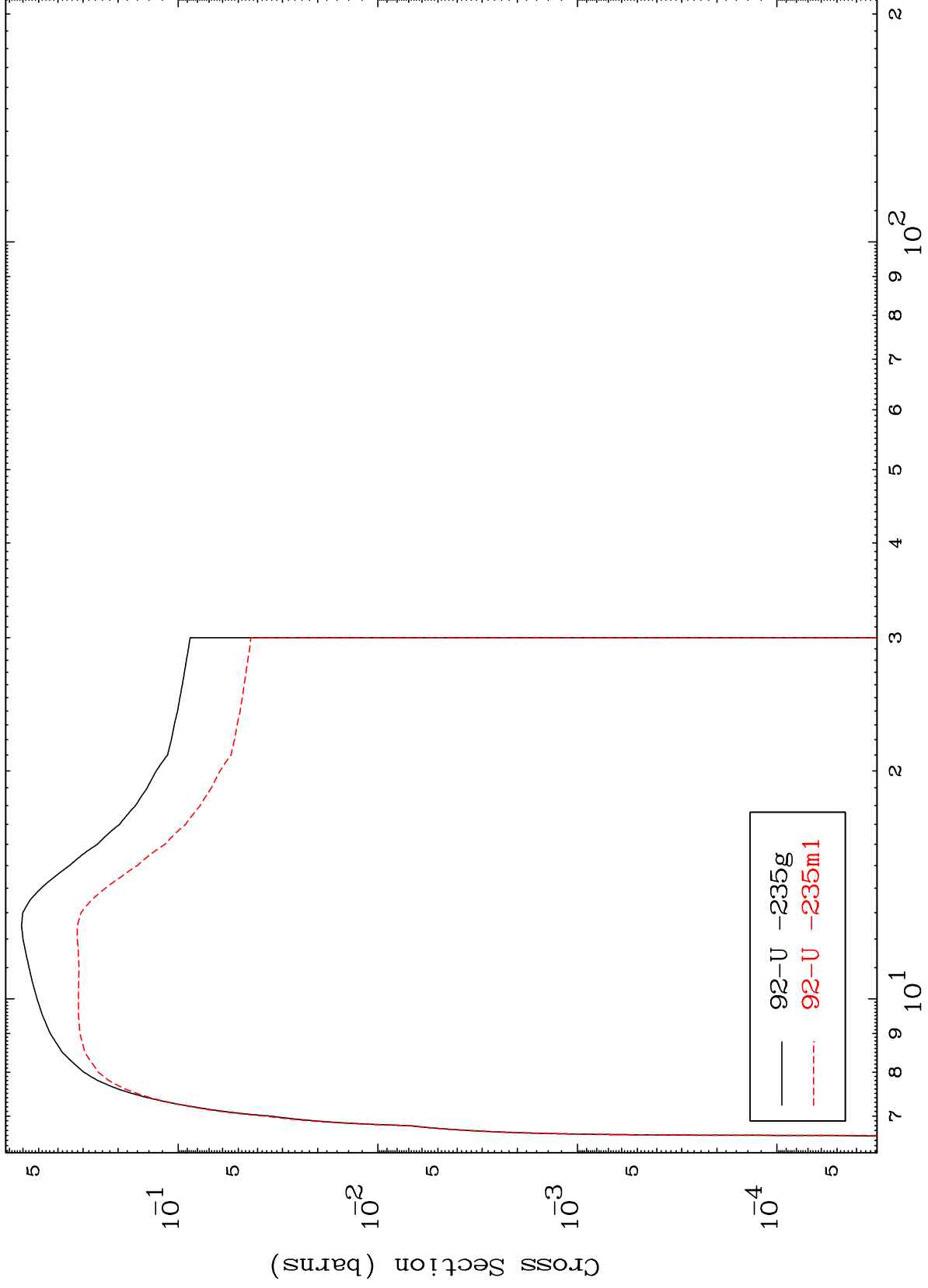
92-U -236



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92-U -236

(n,2n)  
Radionuclide Production Cross Section



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Incident Energy (MeV)

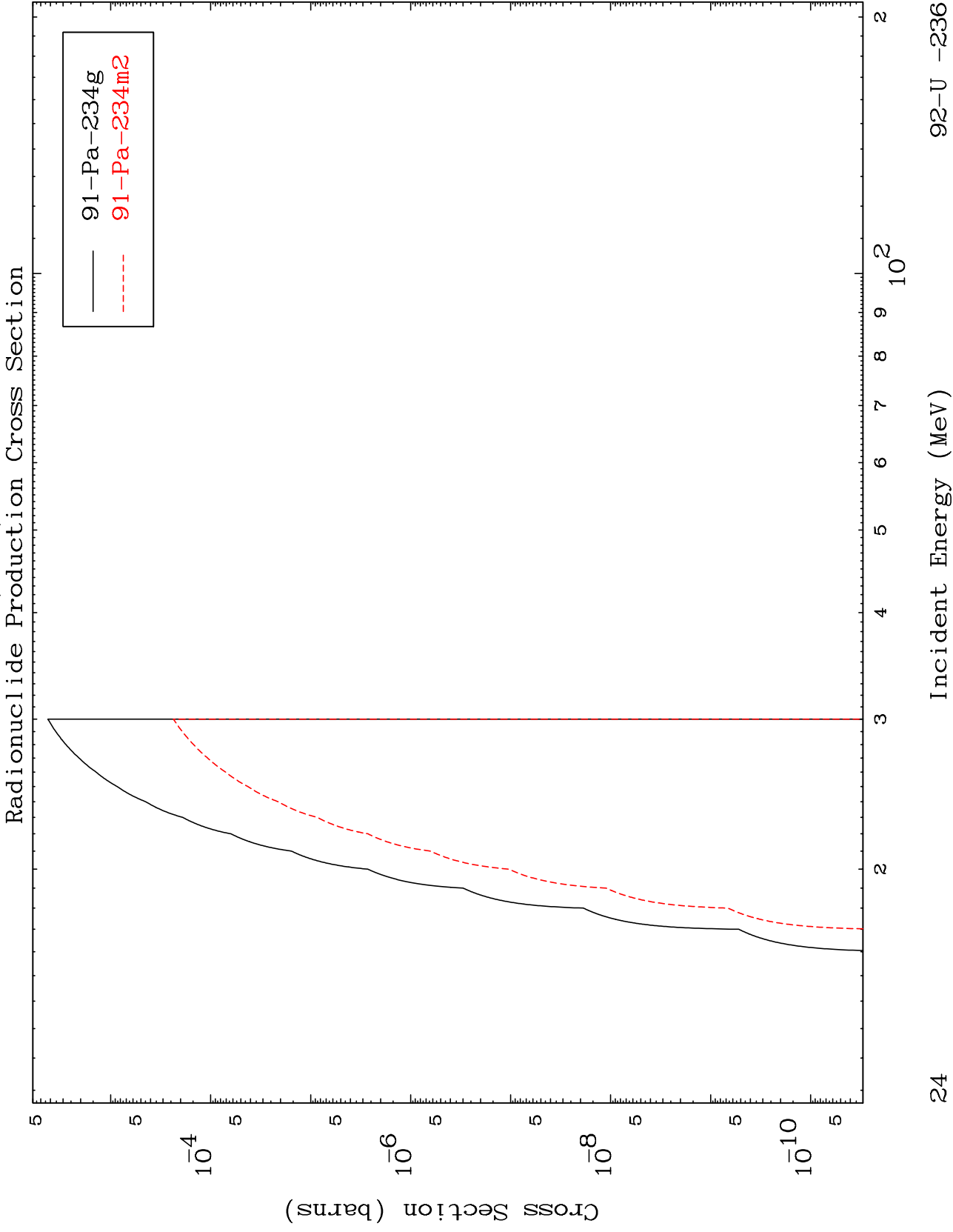
92-U -236



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(n,n') d

92-U -236



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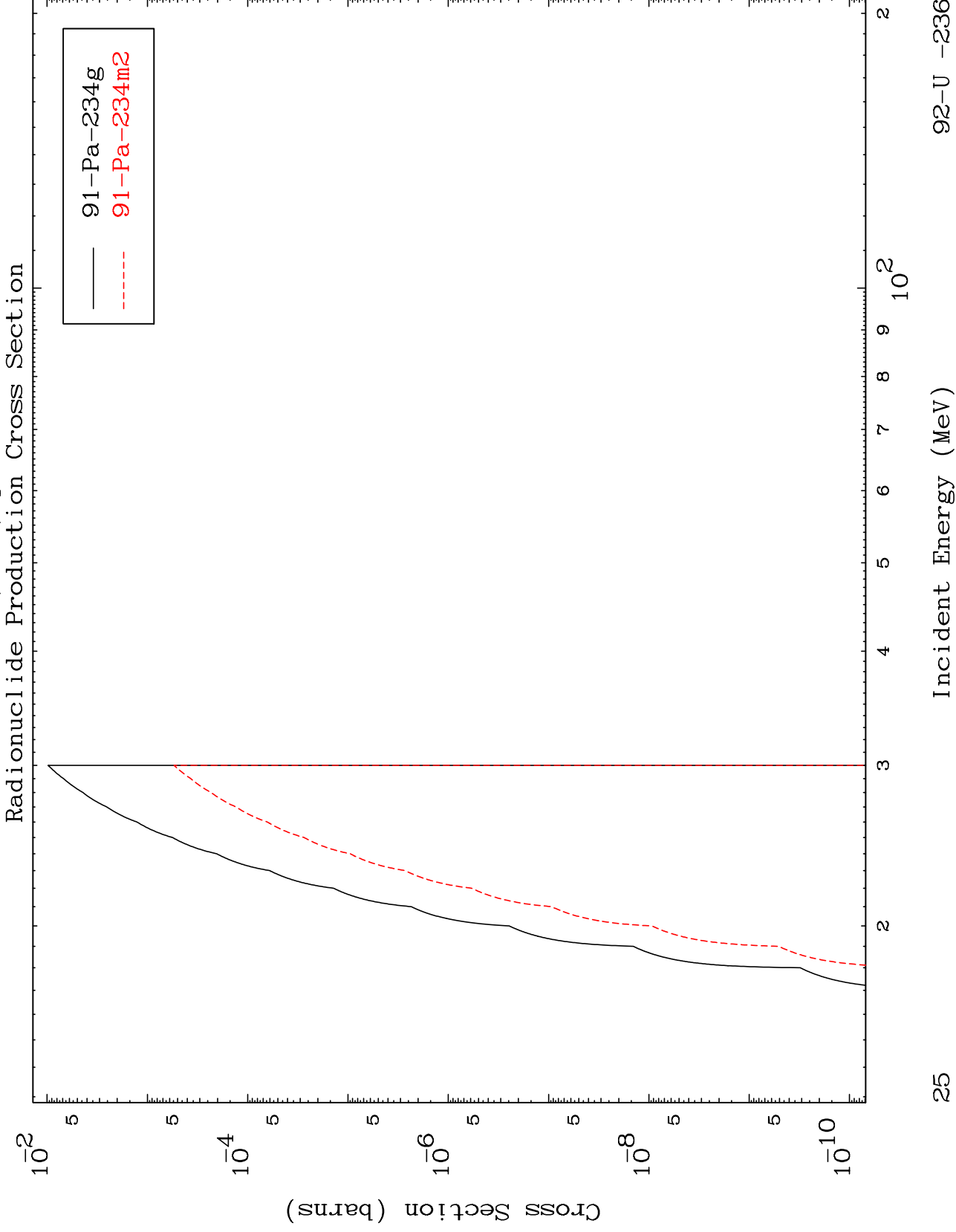
Incident Energy (MeV)

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(n,2n) p

92-U -236



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(n,t)  
Radionuclide Production Cross Section

