

Program EVALPLOT
(Version 2021-1)

by

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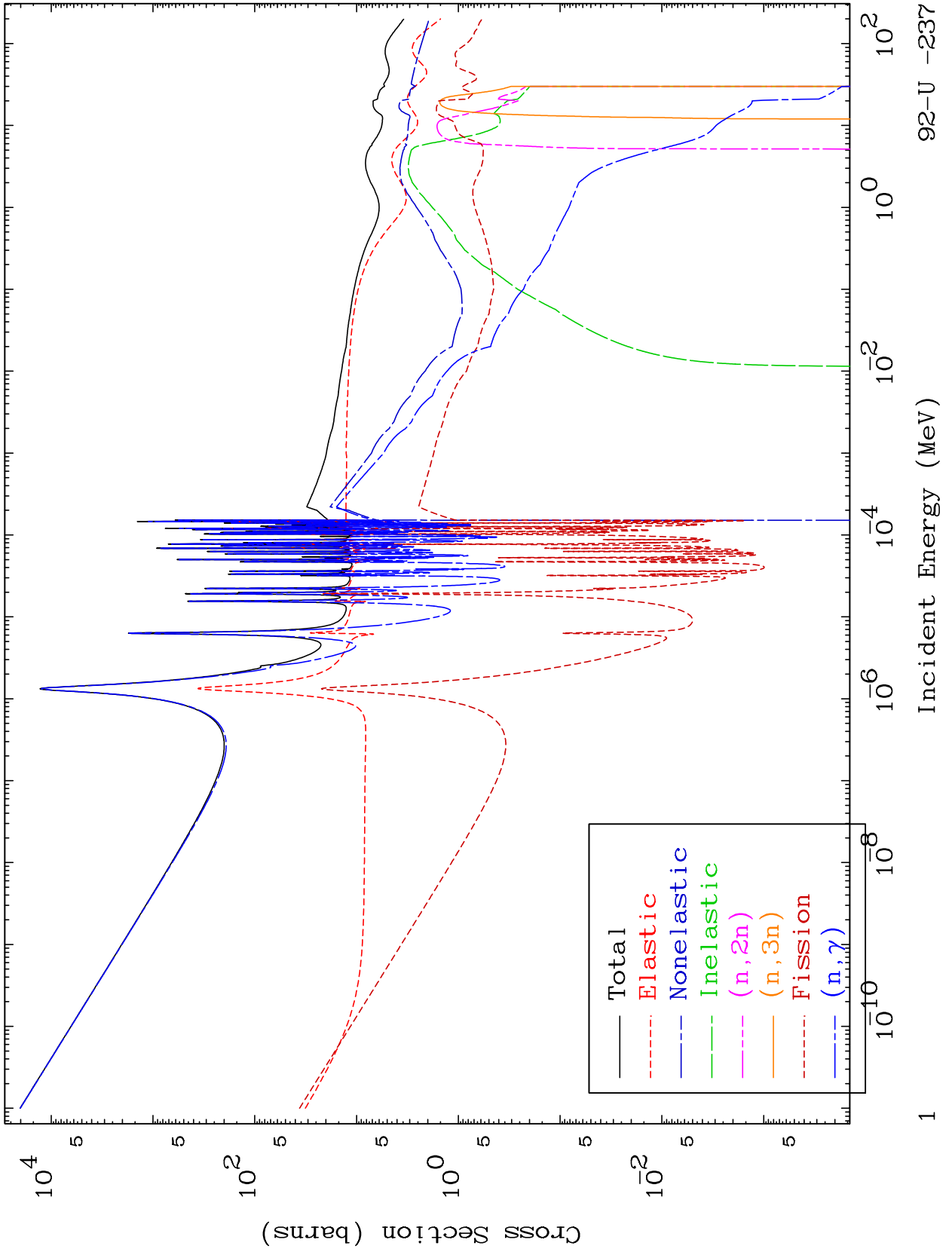
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9234

Neutron Major
293 Kelvin Cross Sections

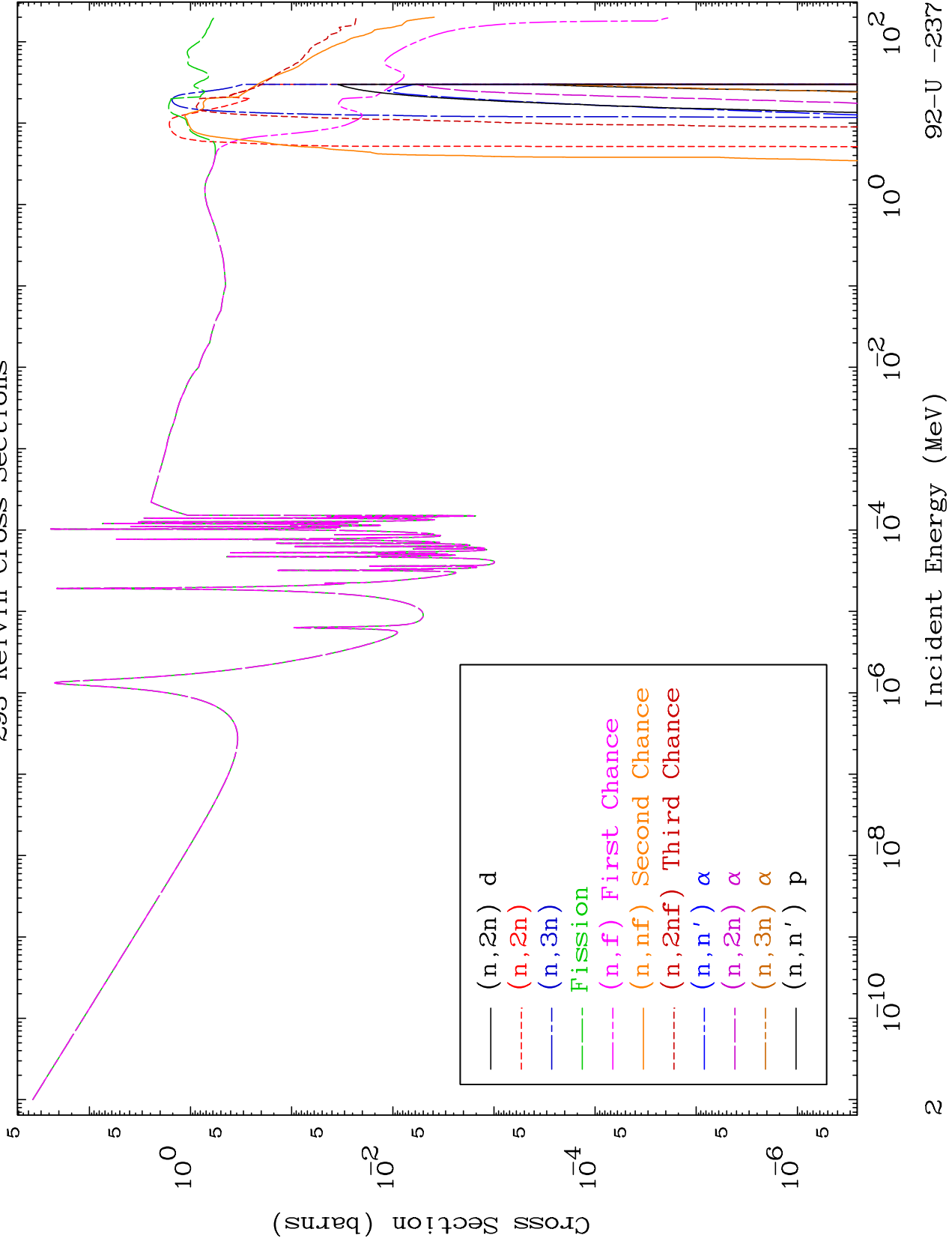
92-U -237

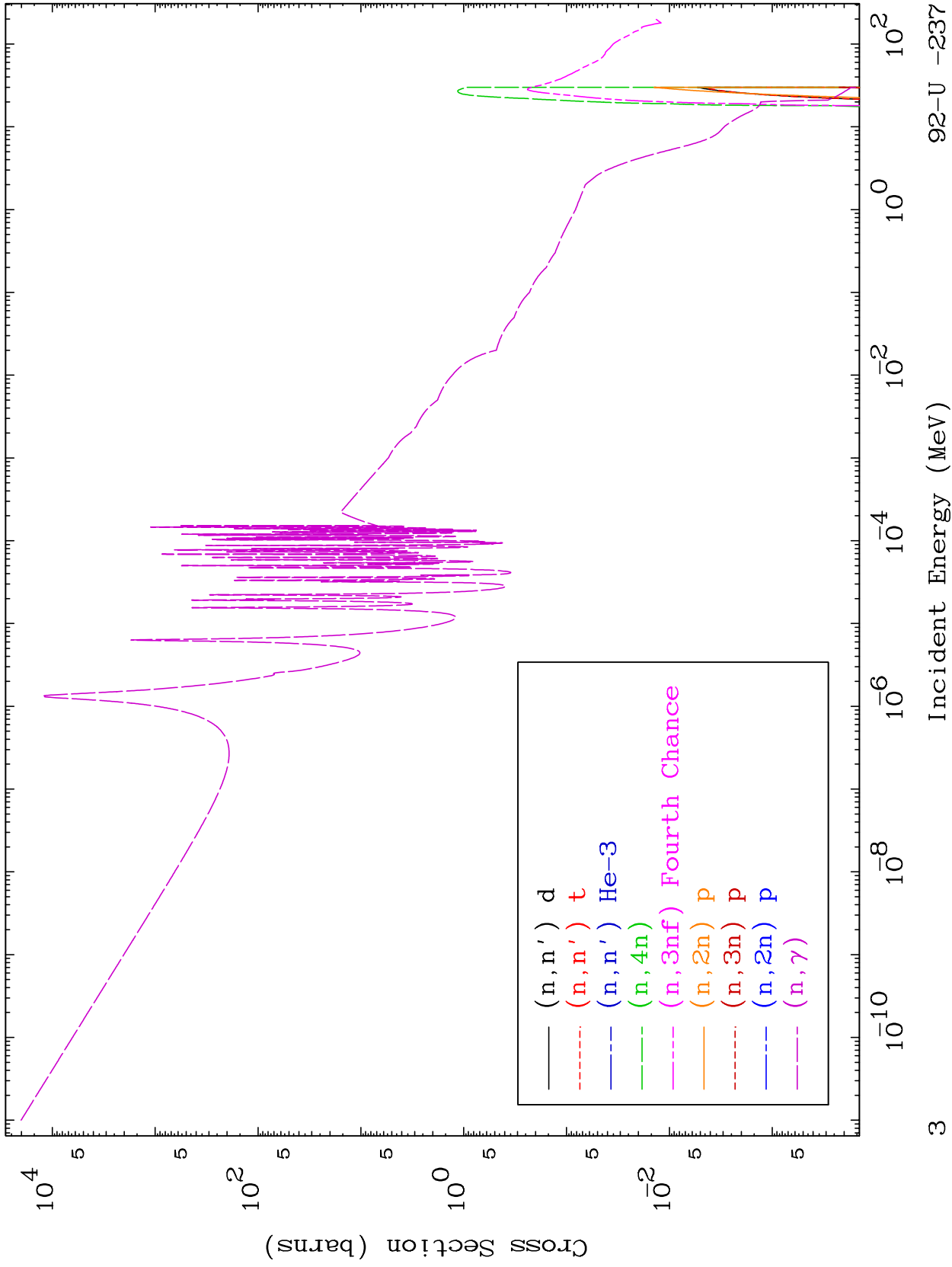


MAT 9234

Neutron Absorption
293 Kelvin Cross Sections

92-U -237

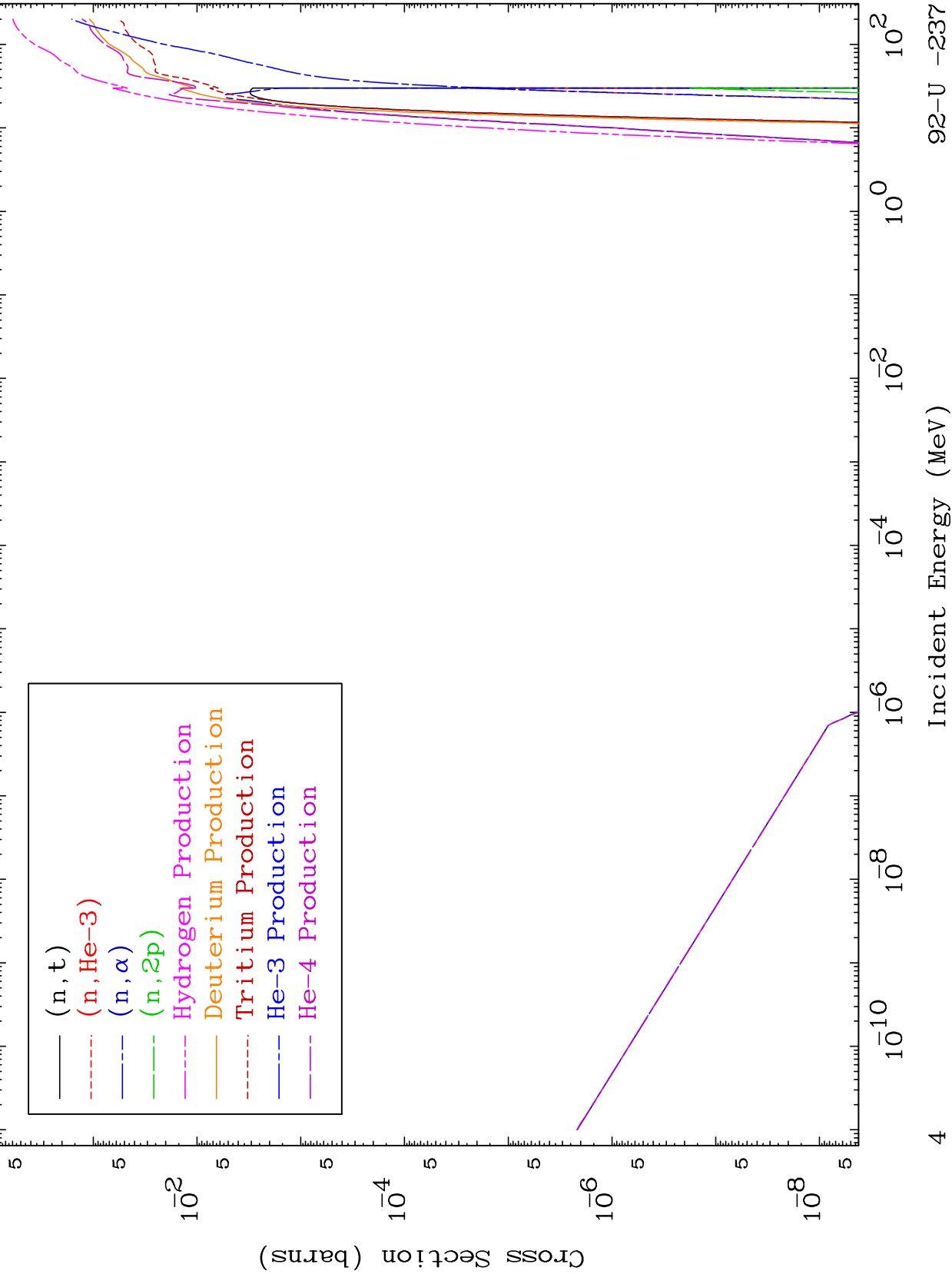




MAT 9234

Neutron Absorption
293 Kelvin Cross Sections

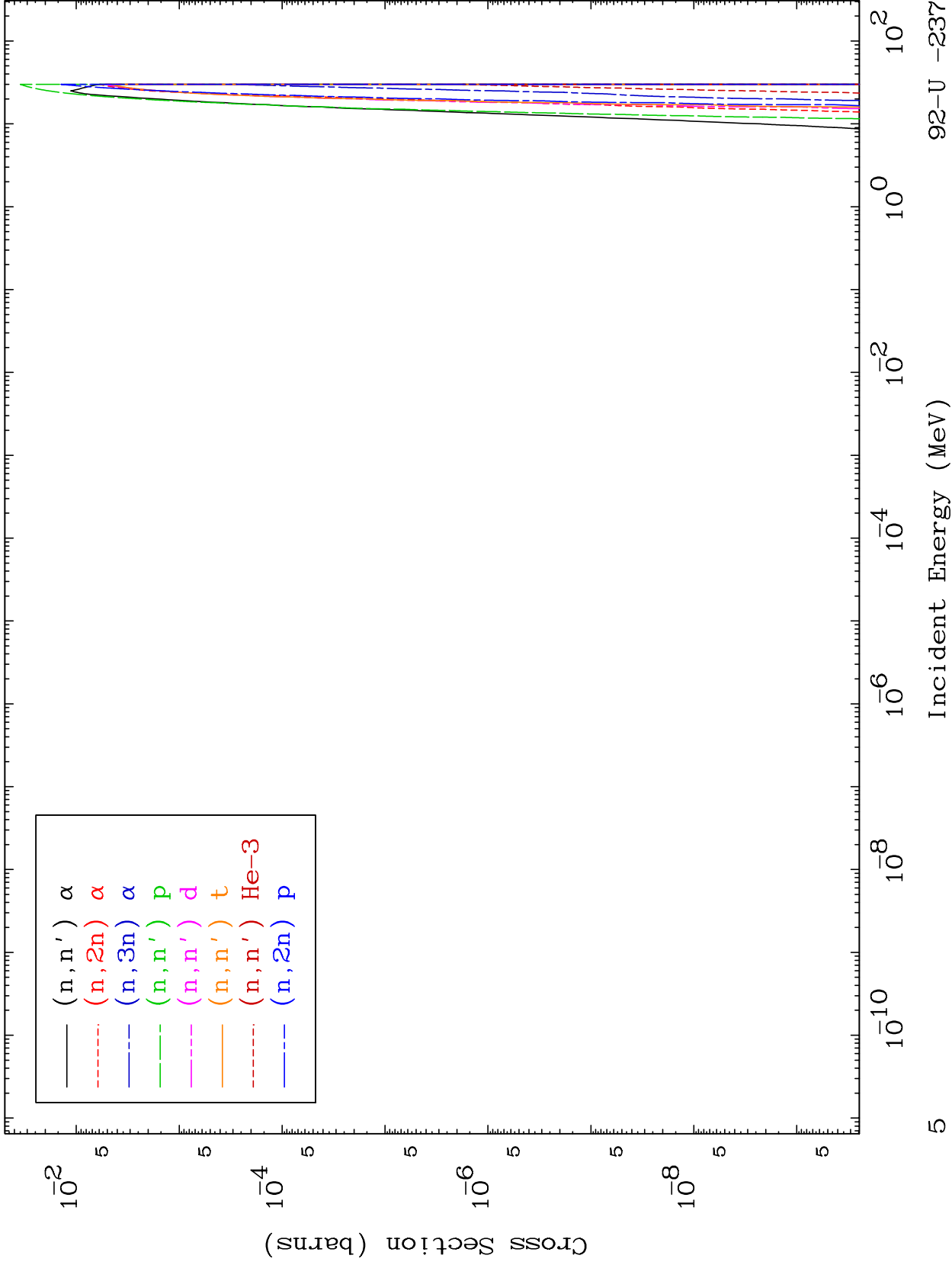
92-U -237



MAT 9234

Charged Particle
293 Kelvin Cross Sections

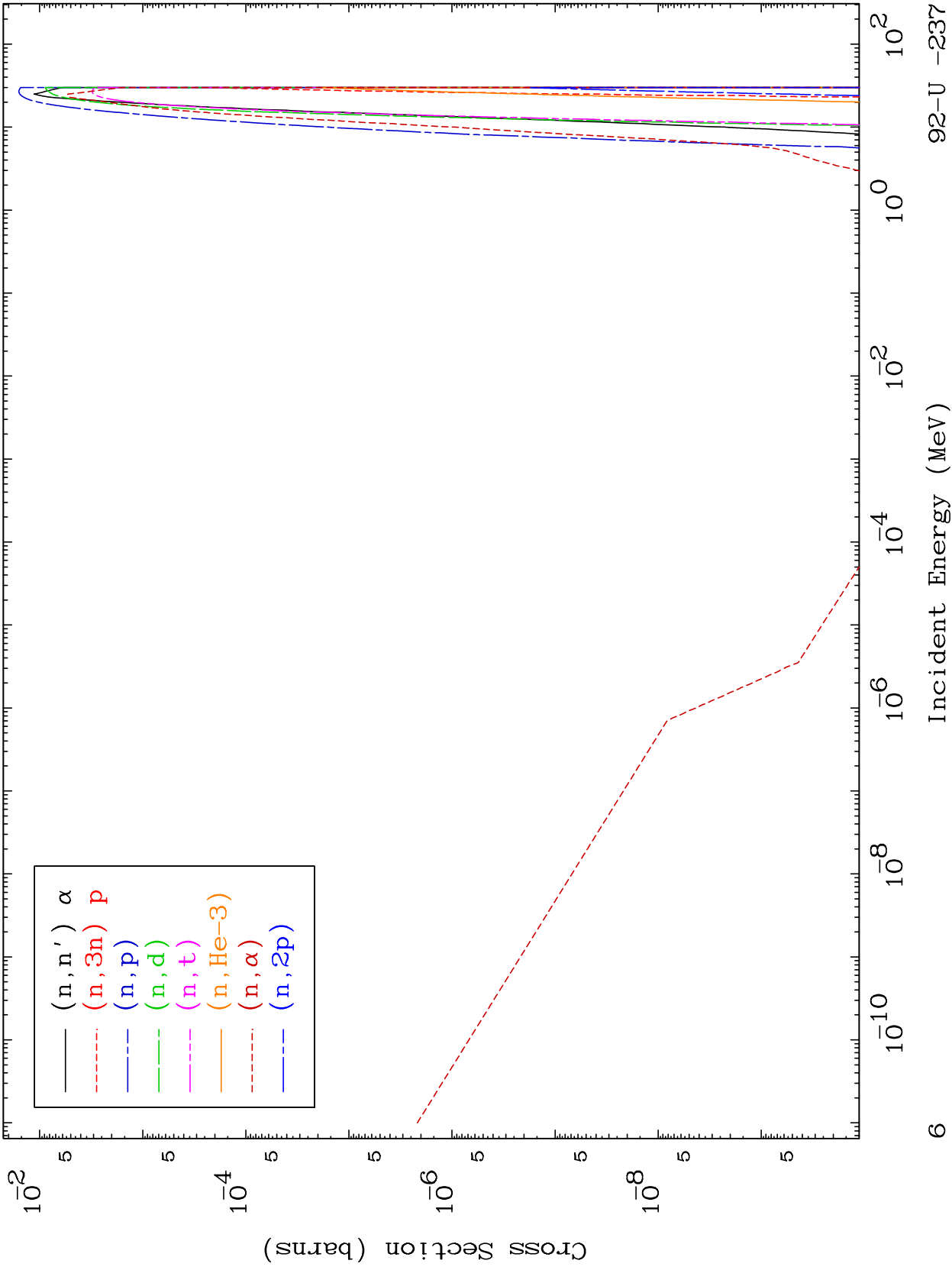
92-U -237

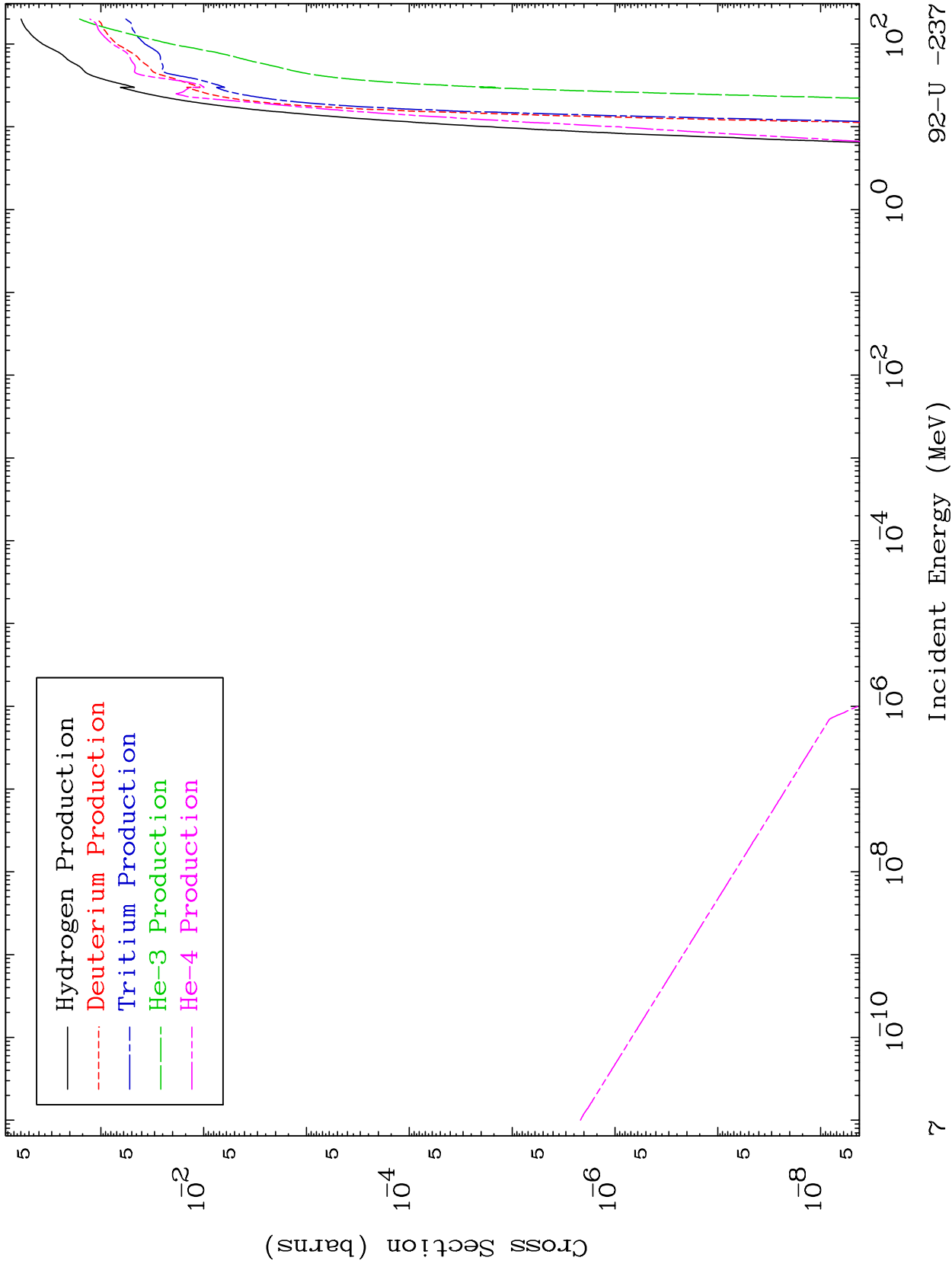


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Charged Particle
293 Kelvin Cross Sections

92-U -237

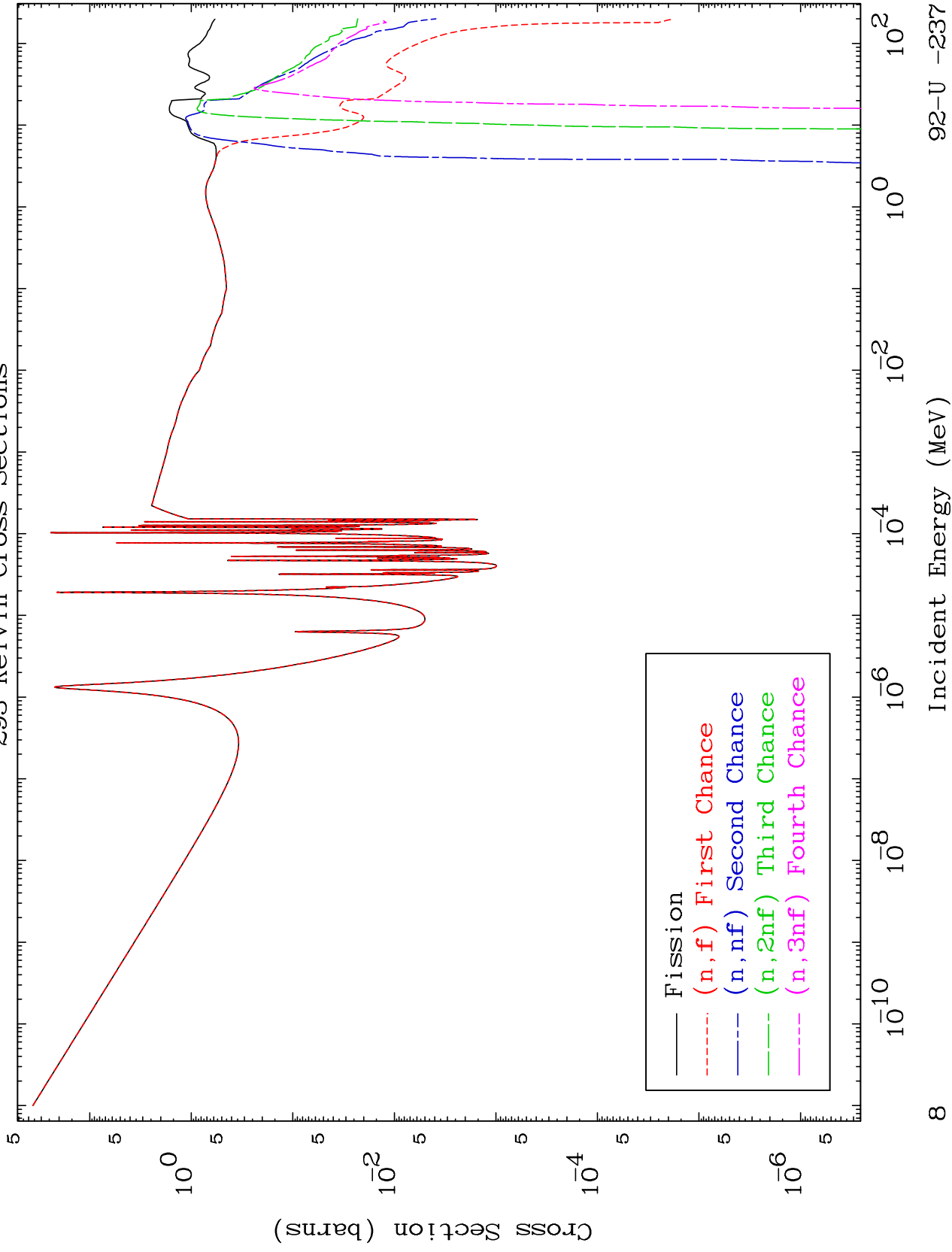




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Fission
293 Kelvin Cross Sections

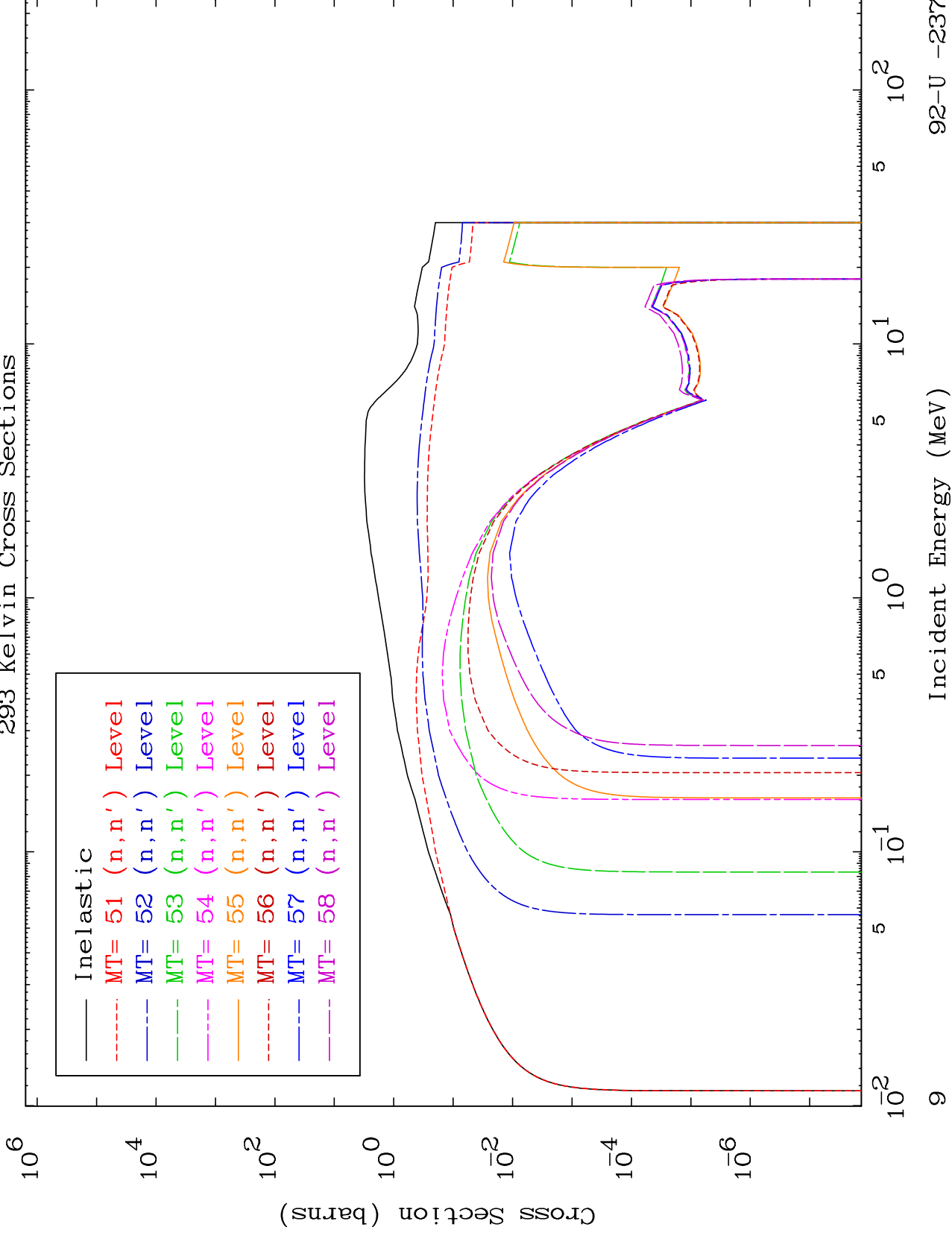
92-U -237



MAT 9234

(n,n') Levels
293 Kelvin Cross Sections

92-U -237



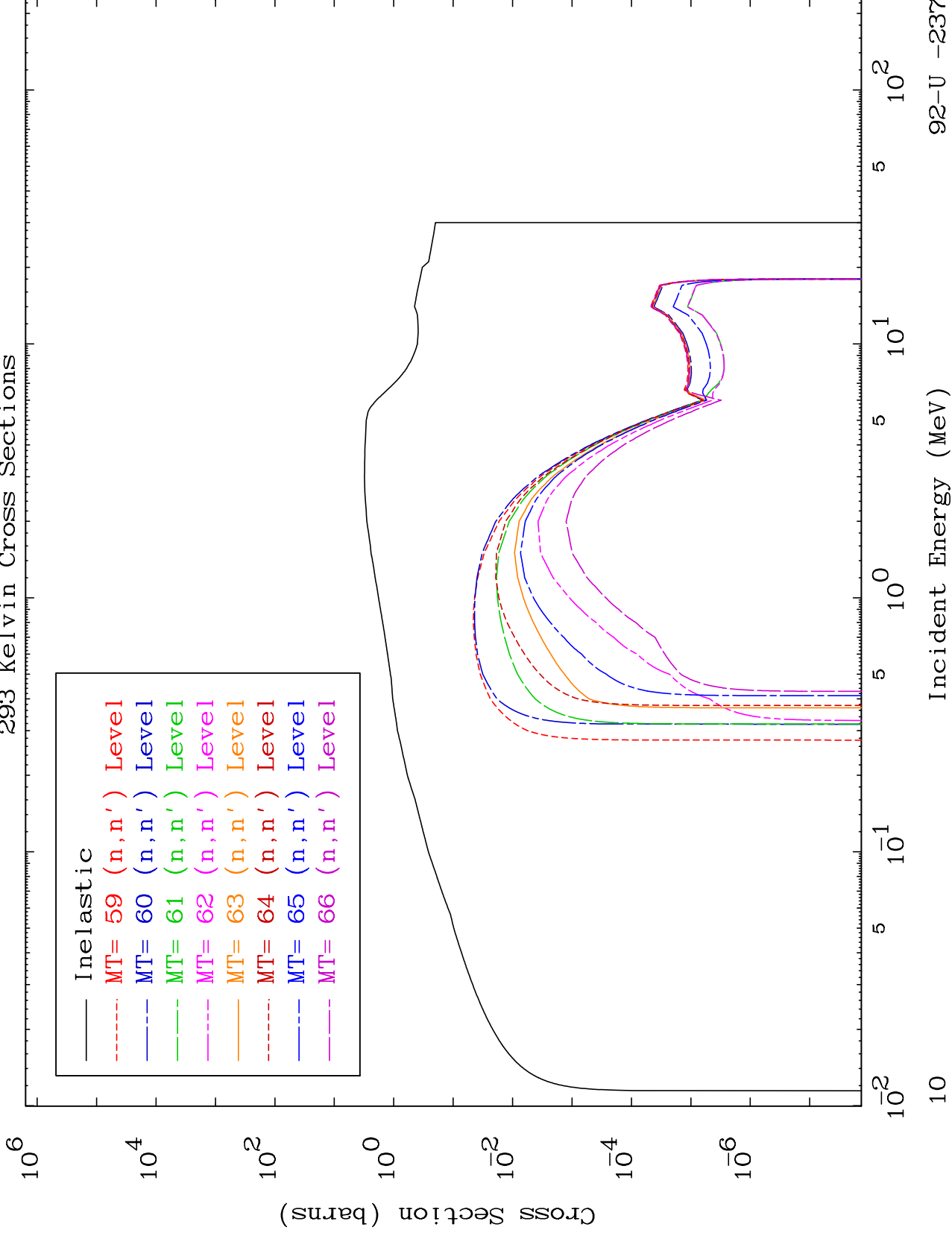
9

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(n,n') Levels
293 Kelvin Cross Sections

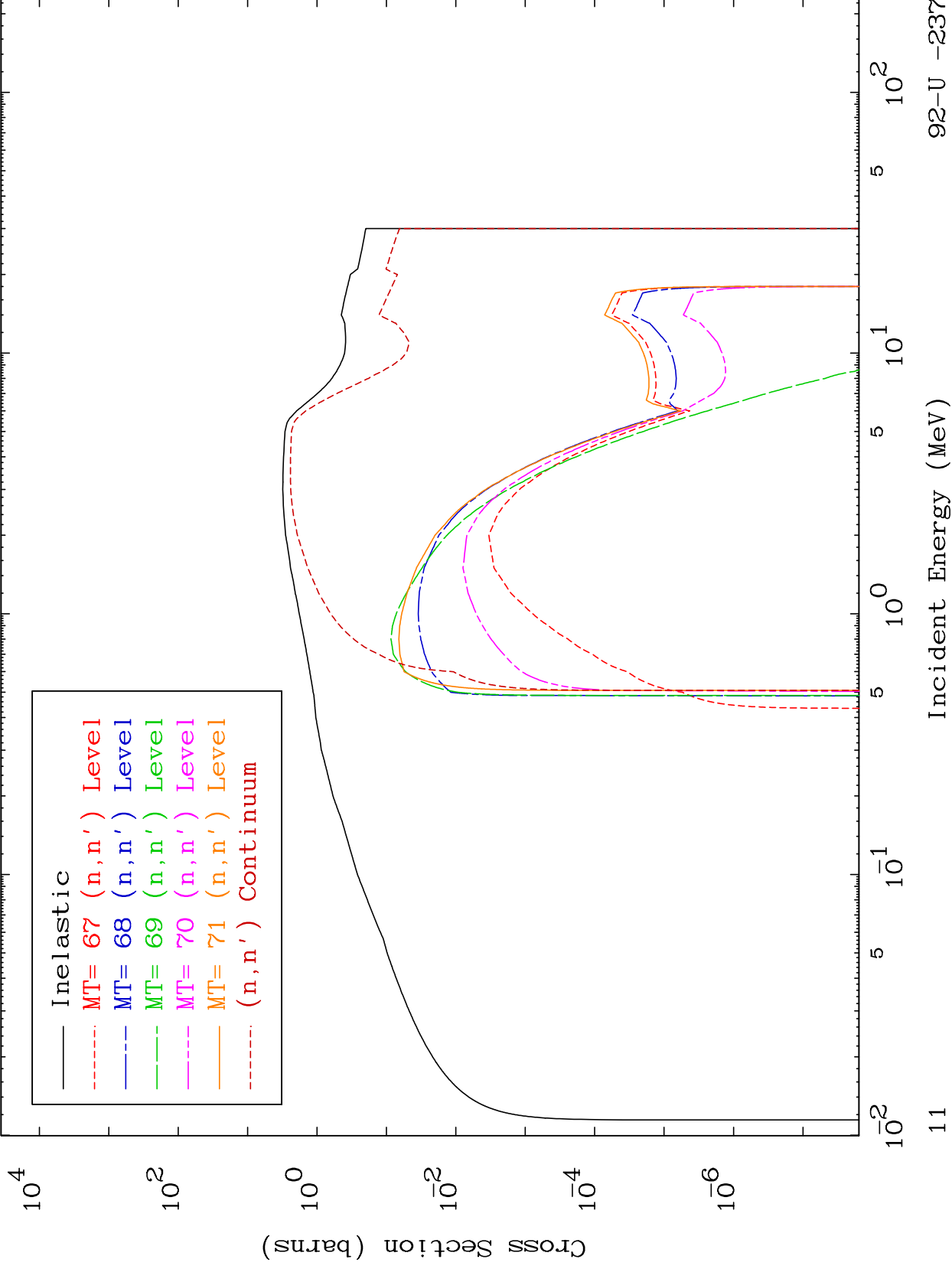
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(n,n') Levels
293 Kelvin Cross Sections

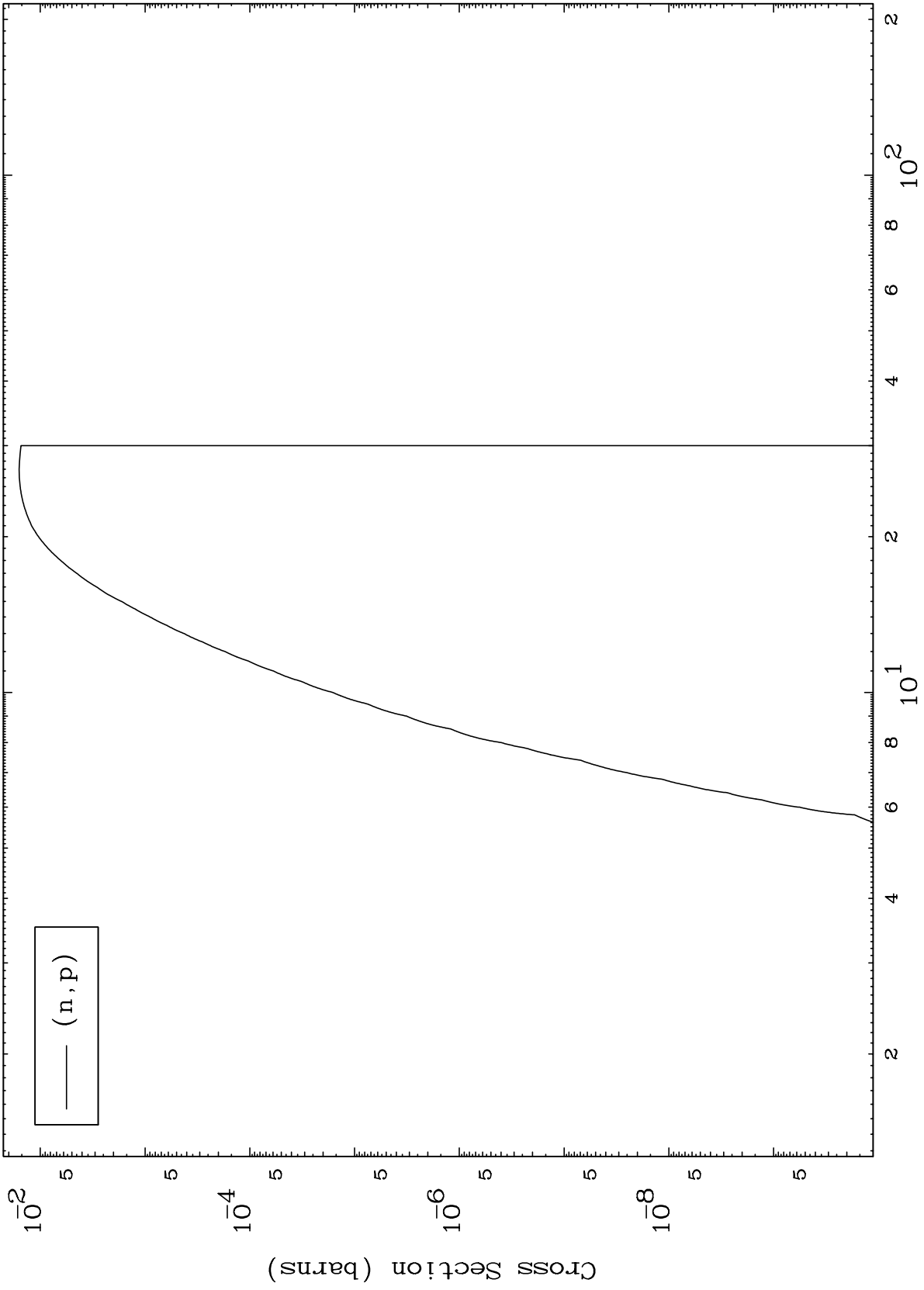
92-U -237



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(n,p) Levels
293 Kelvin Cross Sections

92-U -237



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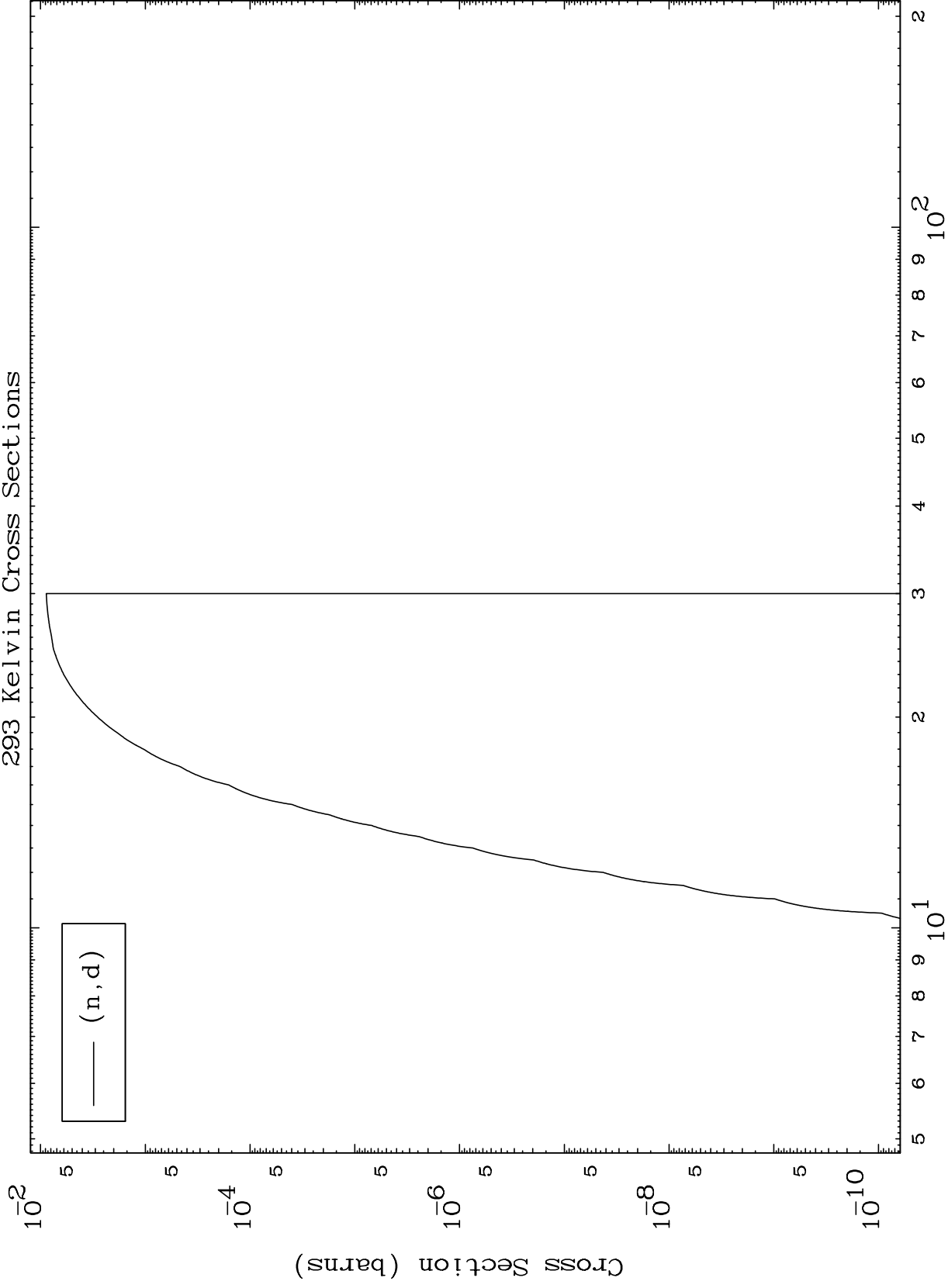
Incident Energy (MeV)

92-U -237

MAT 9234

(n,d) Levels
293 Kelvin Cross Sections

92-U -237



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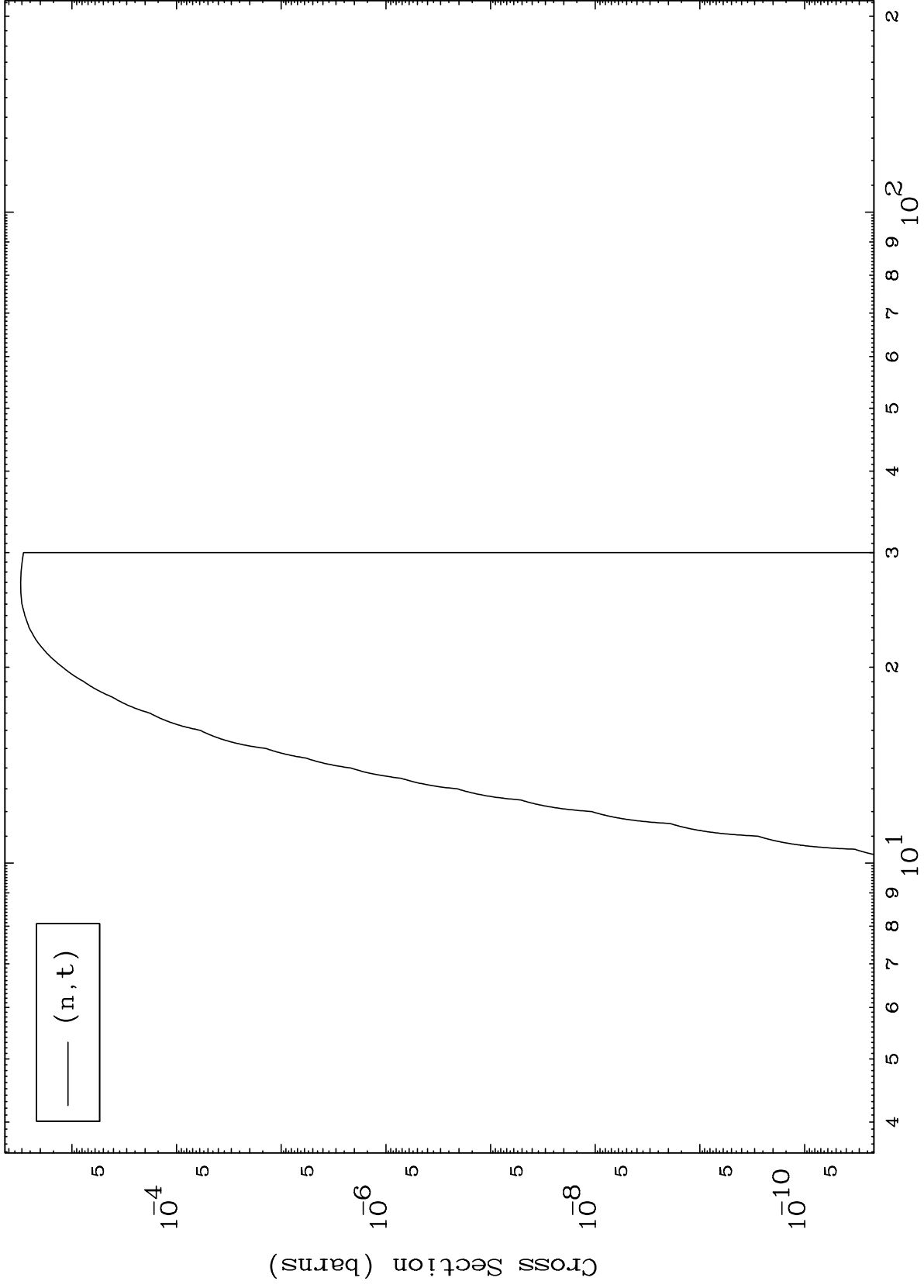
Incident Energy (MeV)

92-U -237

MAT 9234

(n,t) Levels
293 Kelvin Cross Sections

92-U -237



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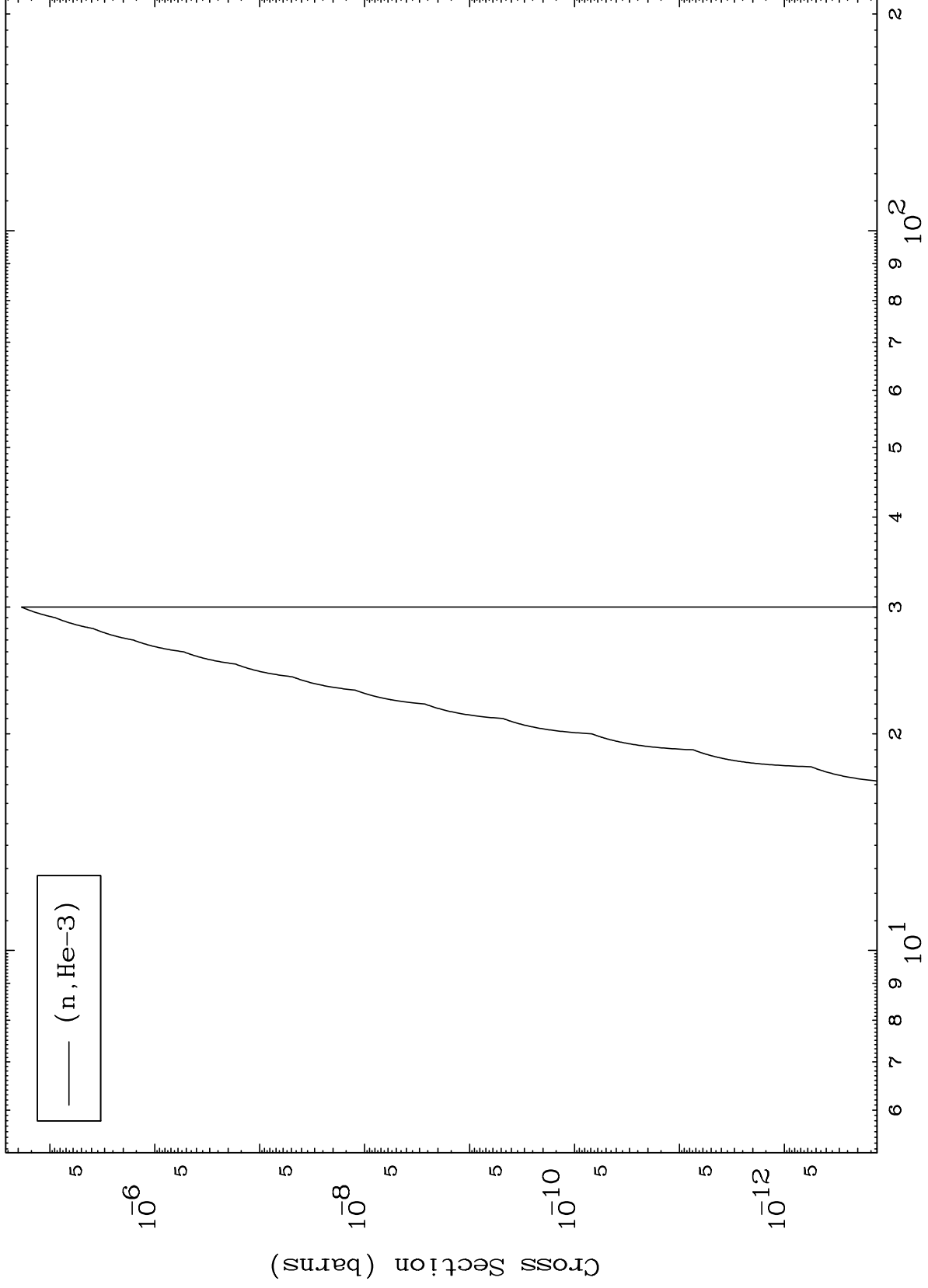
Incident Energy (MeV)

92-U -237

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(n,He3) Levels
293 Kelvin Cross Sections

92-U -237



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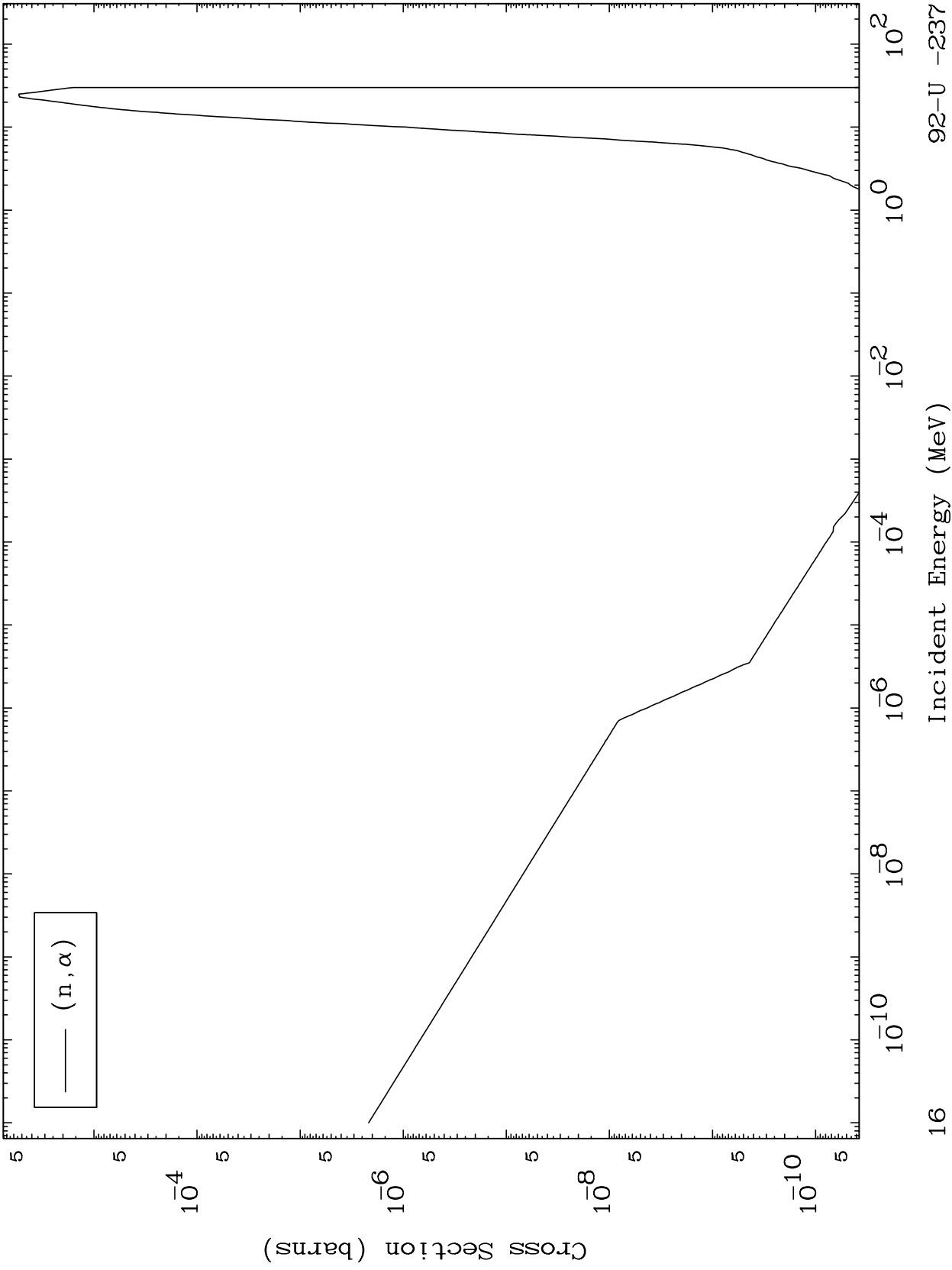
Incident Energy (MeV)

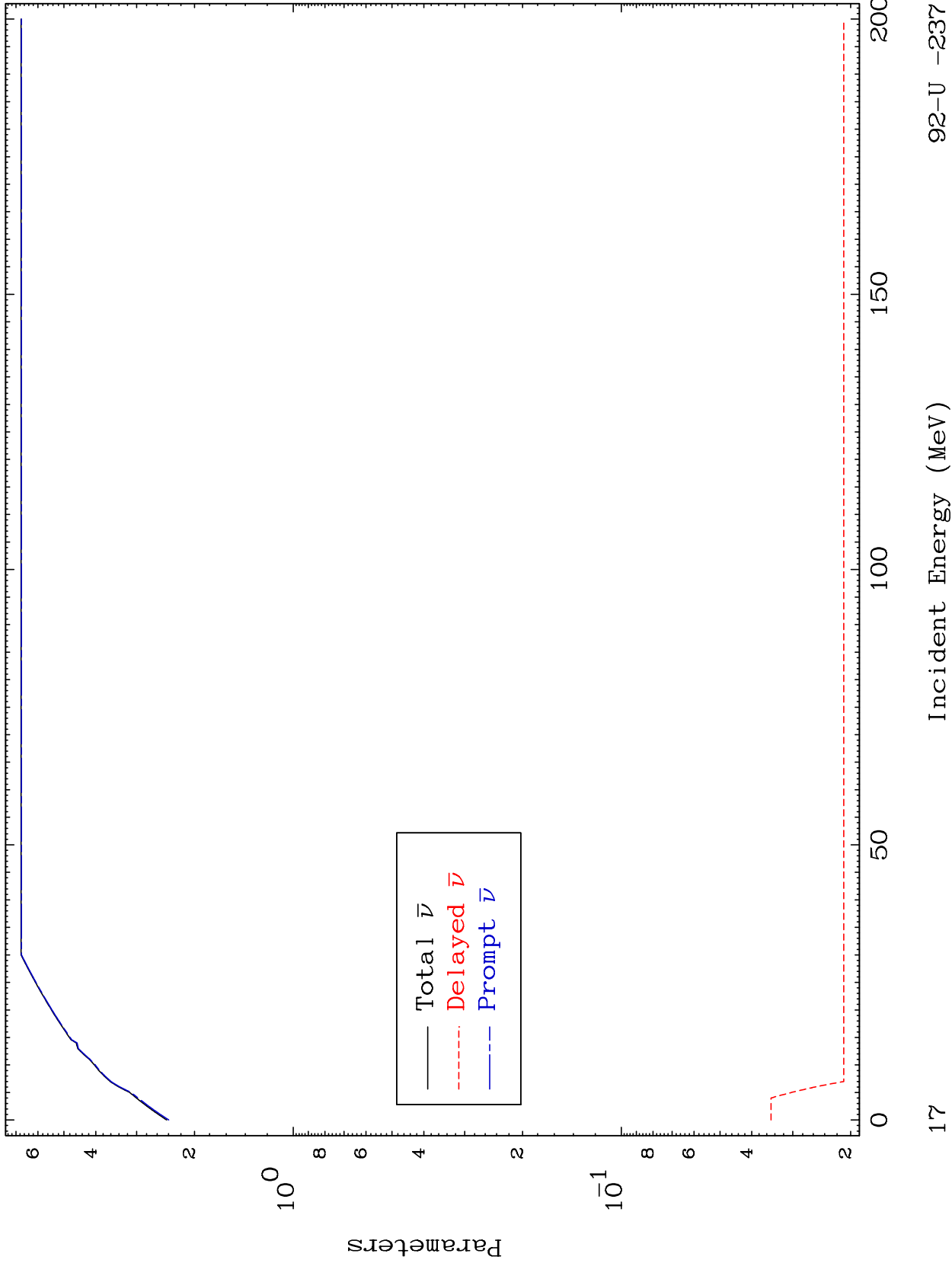
92-U -237

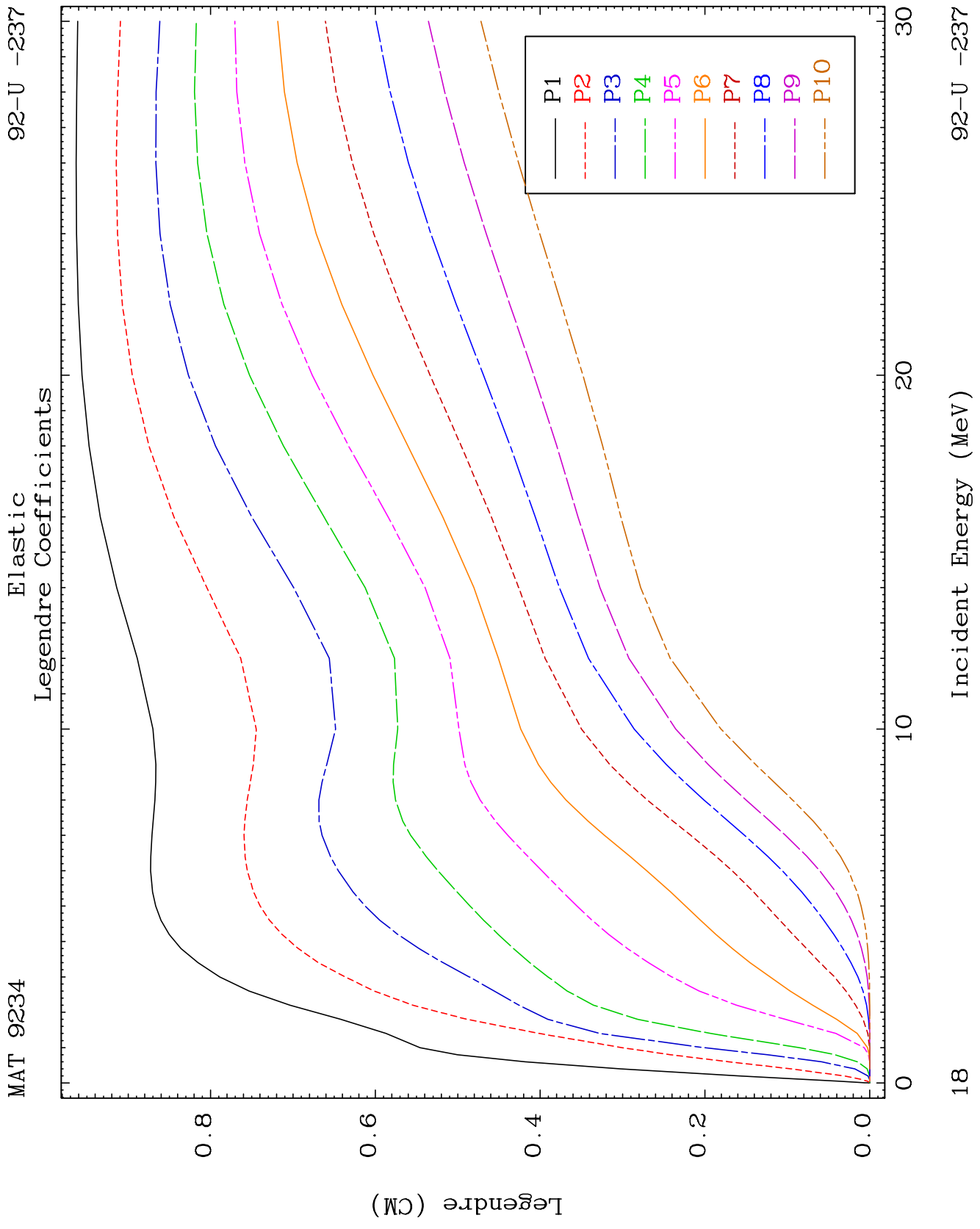
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(n, α) Levels
293 Kelvin Cross Sections

92-U -237



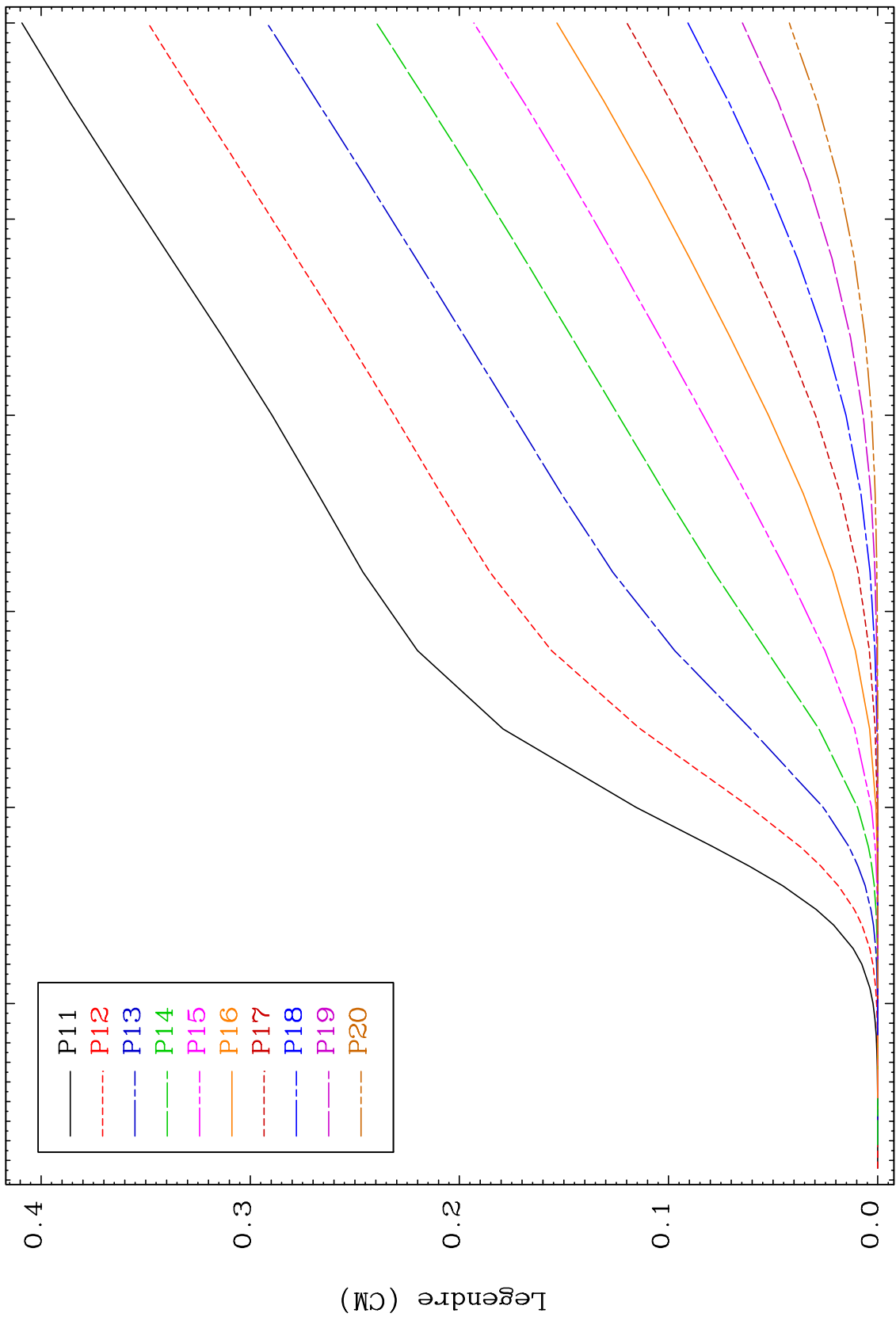




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Elastic Legendre Coefficients

92-U -237



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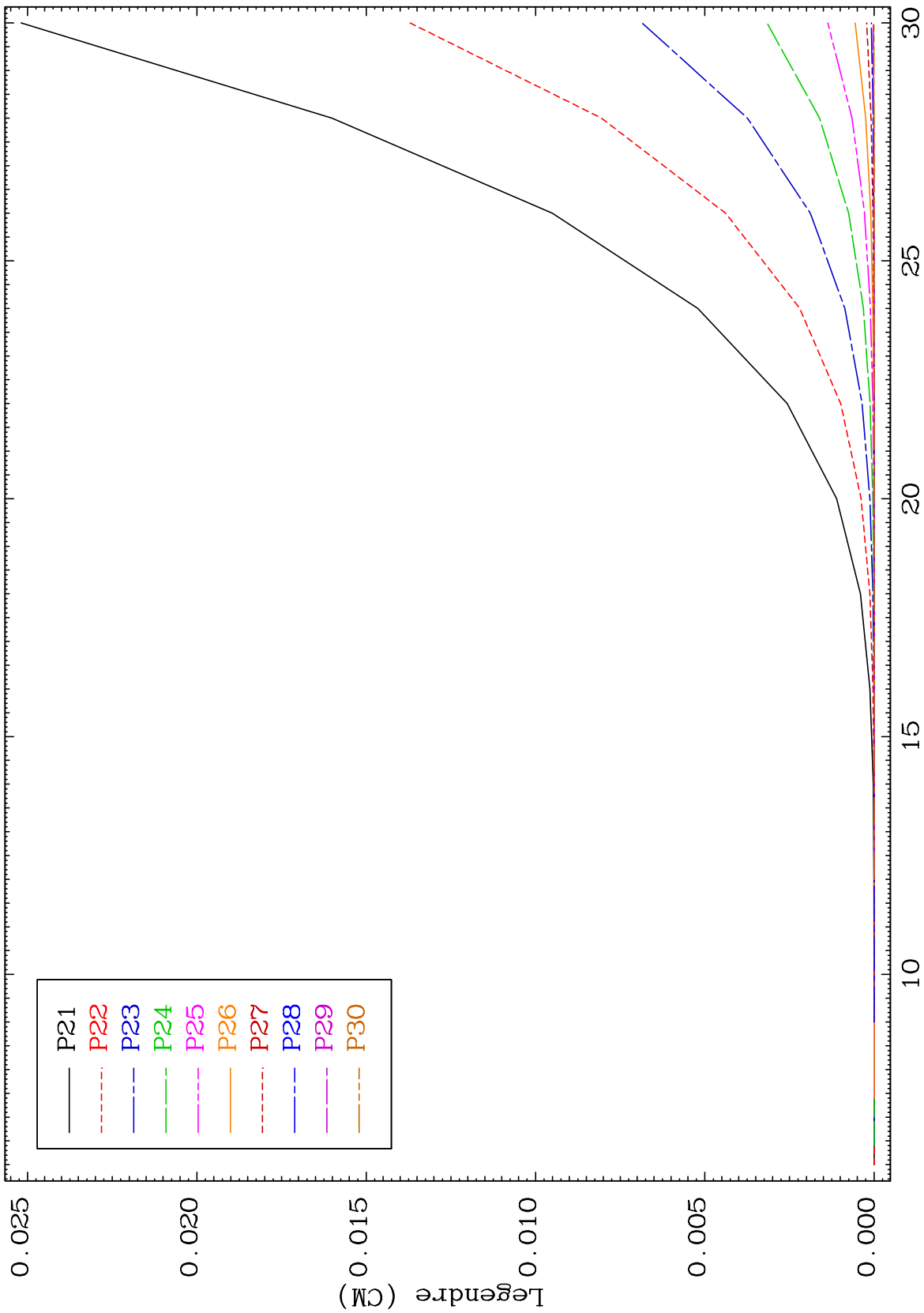
Incident Energy (MeV)

92-U -237

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Elastic Legendre Coefficients

92-U -237



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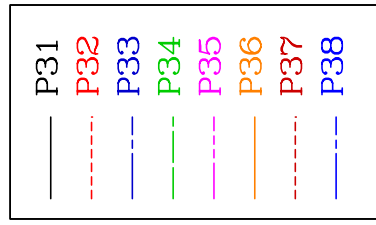
Incident Energy (MeV)

92-U -237

MAT 9234

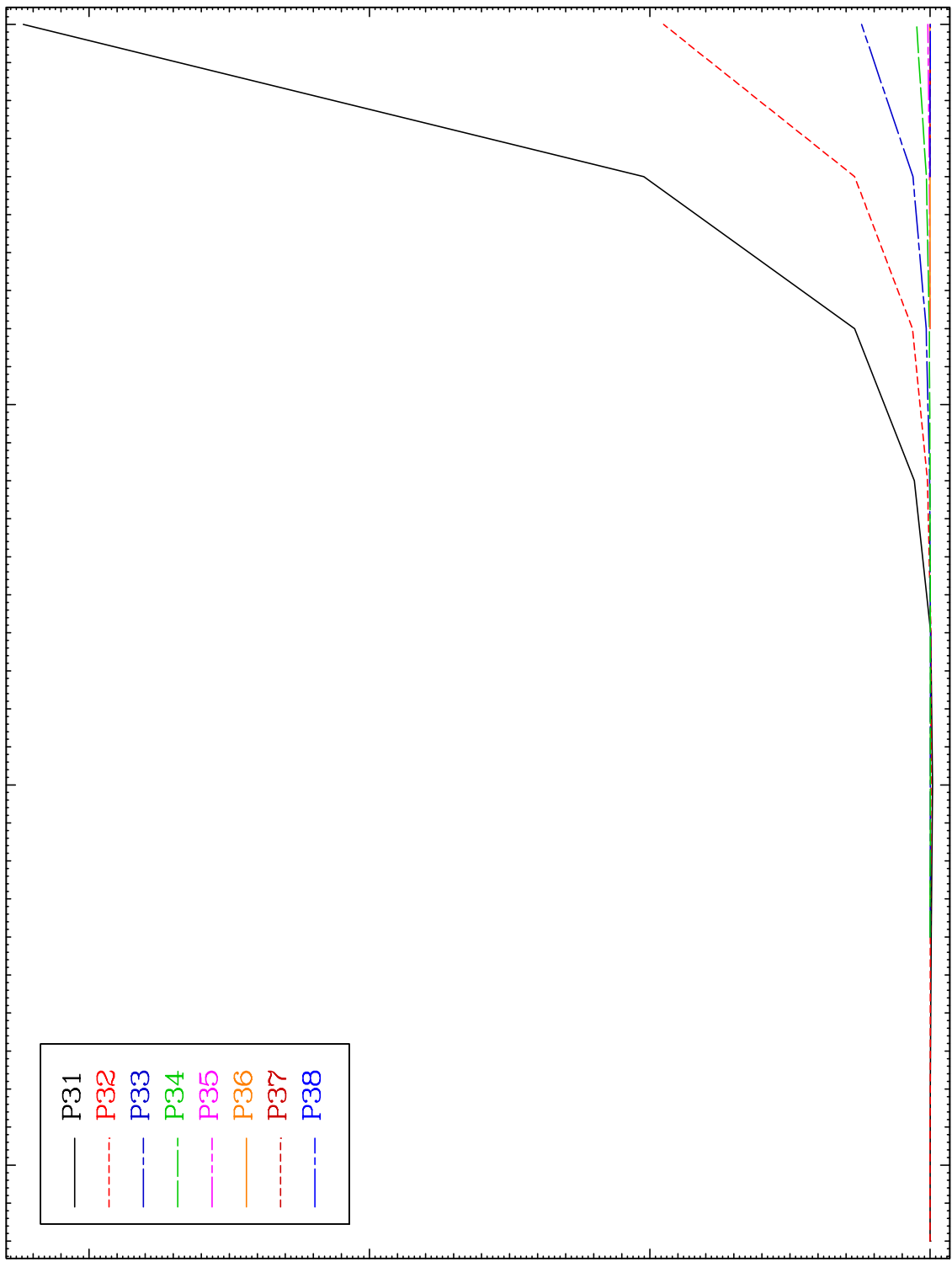
Elastic
Legendre Coefficients

92-U -237



$\times 10^{-6}$

Legendre (CM)

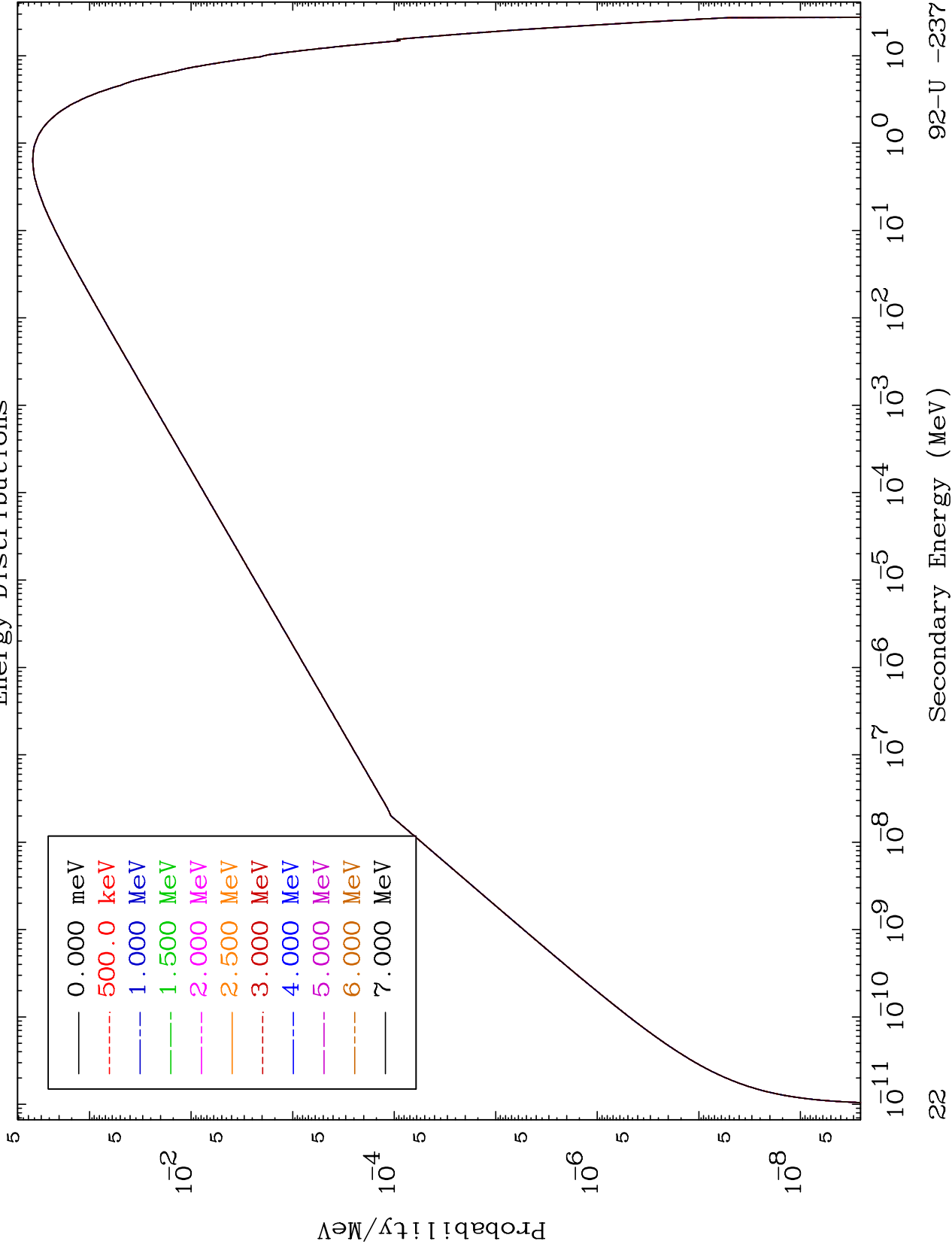


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Incident Energy (MeV)

92-U -237

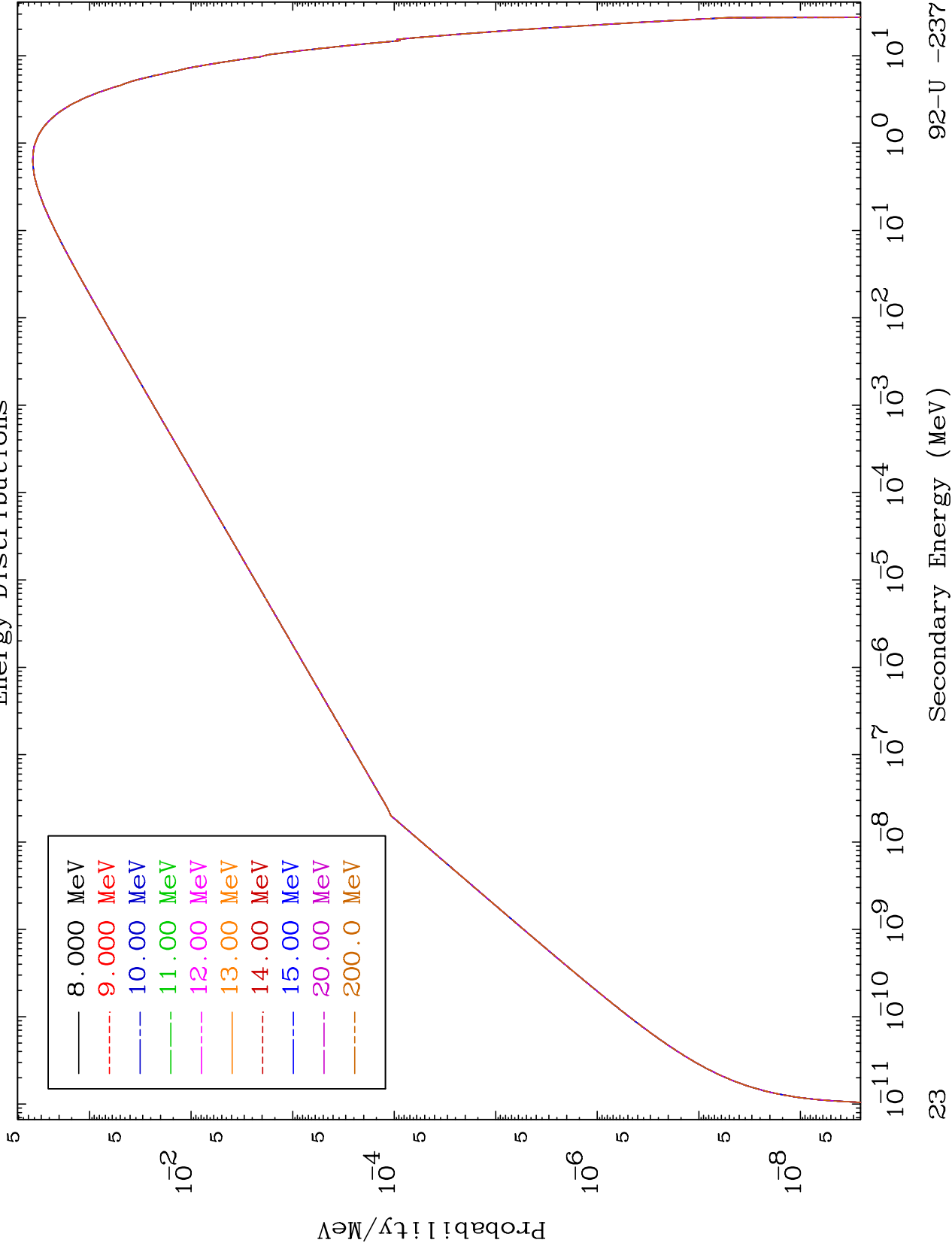
Fission Energy Distributions



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Fission Energy Distributions

92-U -237



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Secondary Energy (MeV)

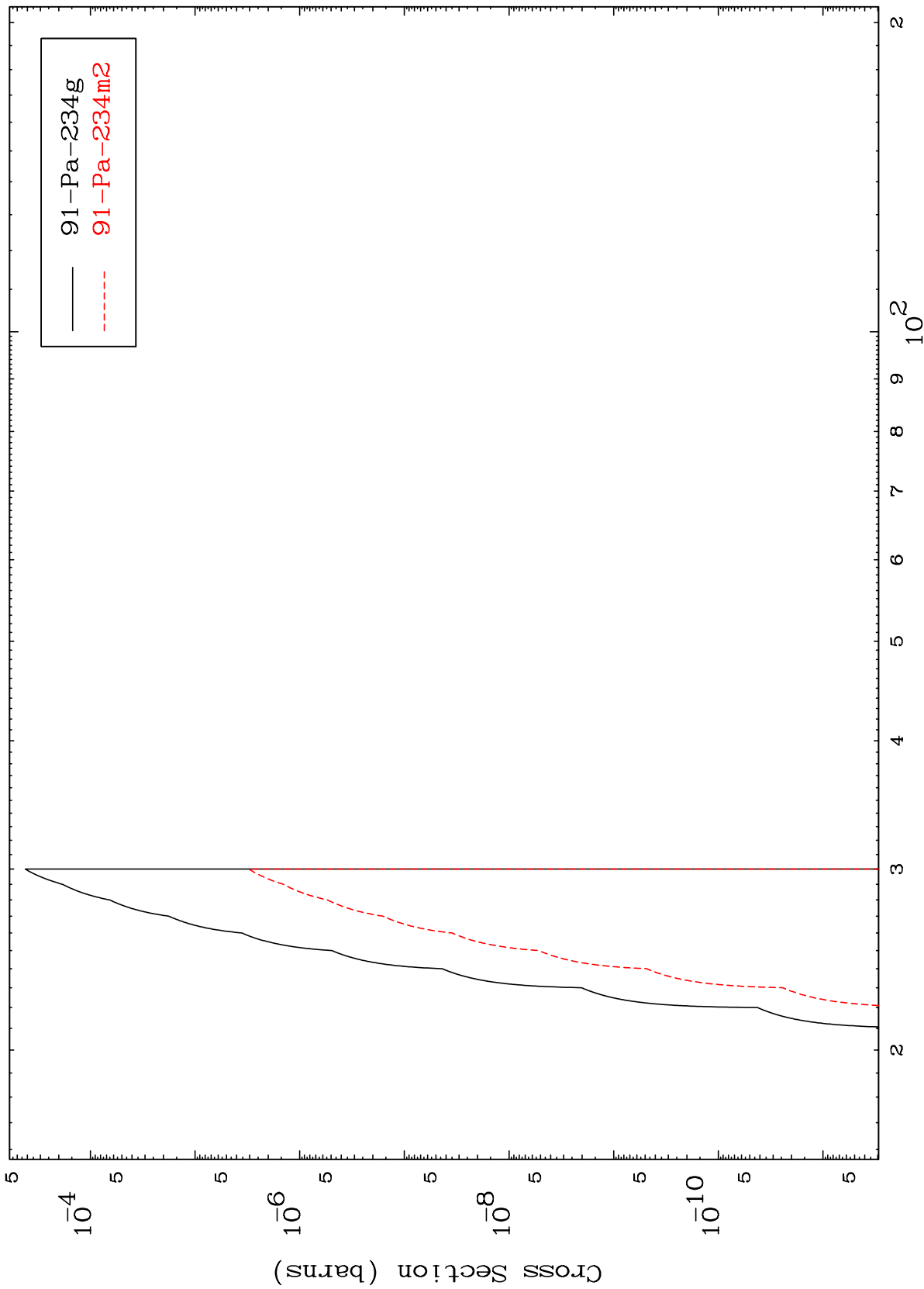
23

MAT 92334

(n,2n) d

92-U -237

Radionuclide Production Cross Section



24

Incident Energy (MeV)

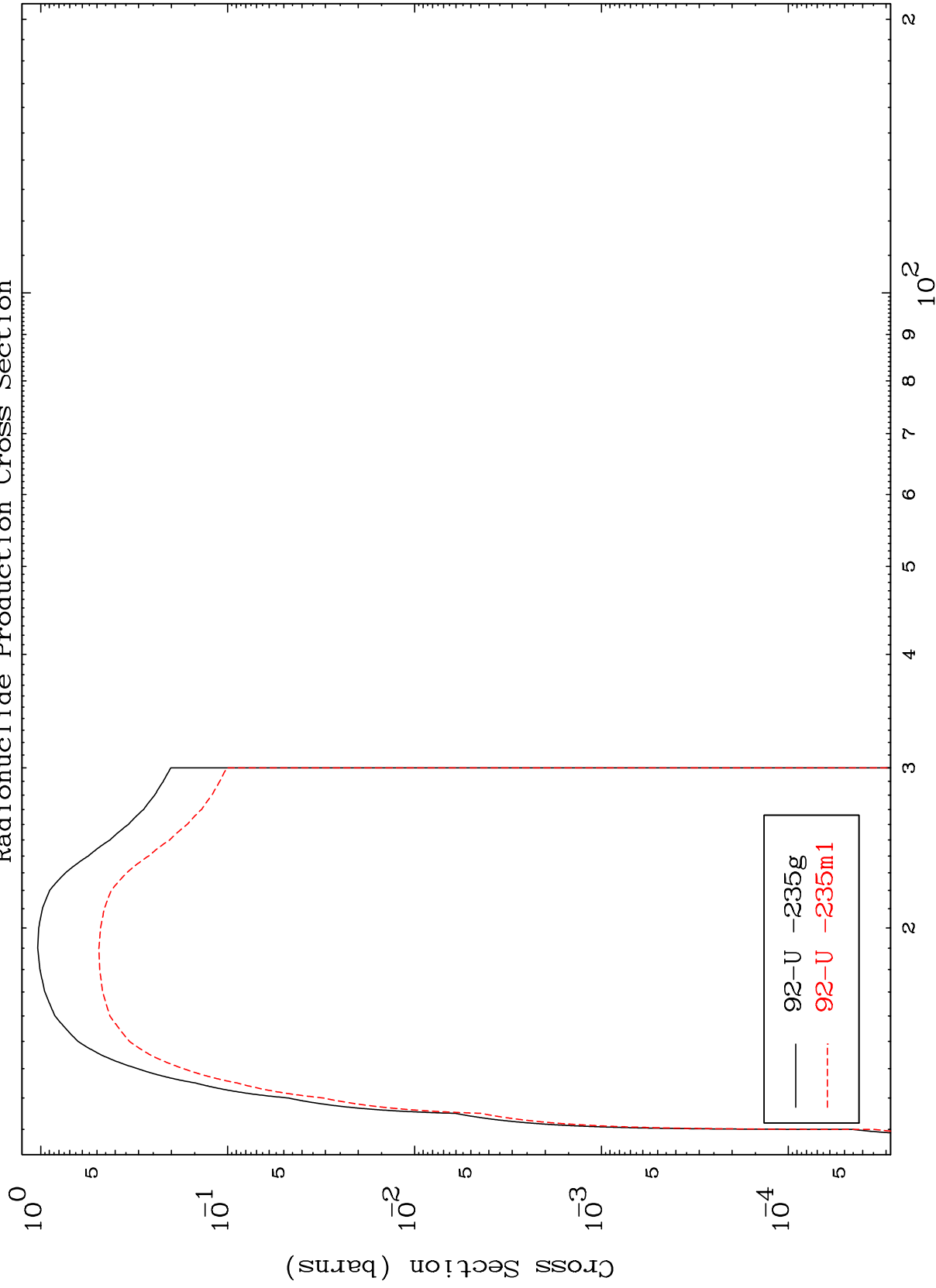
92-U -237

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(n,3n)

92-U -237

Radionuclide Production Cross Section



25

Incident Energy (MeV)

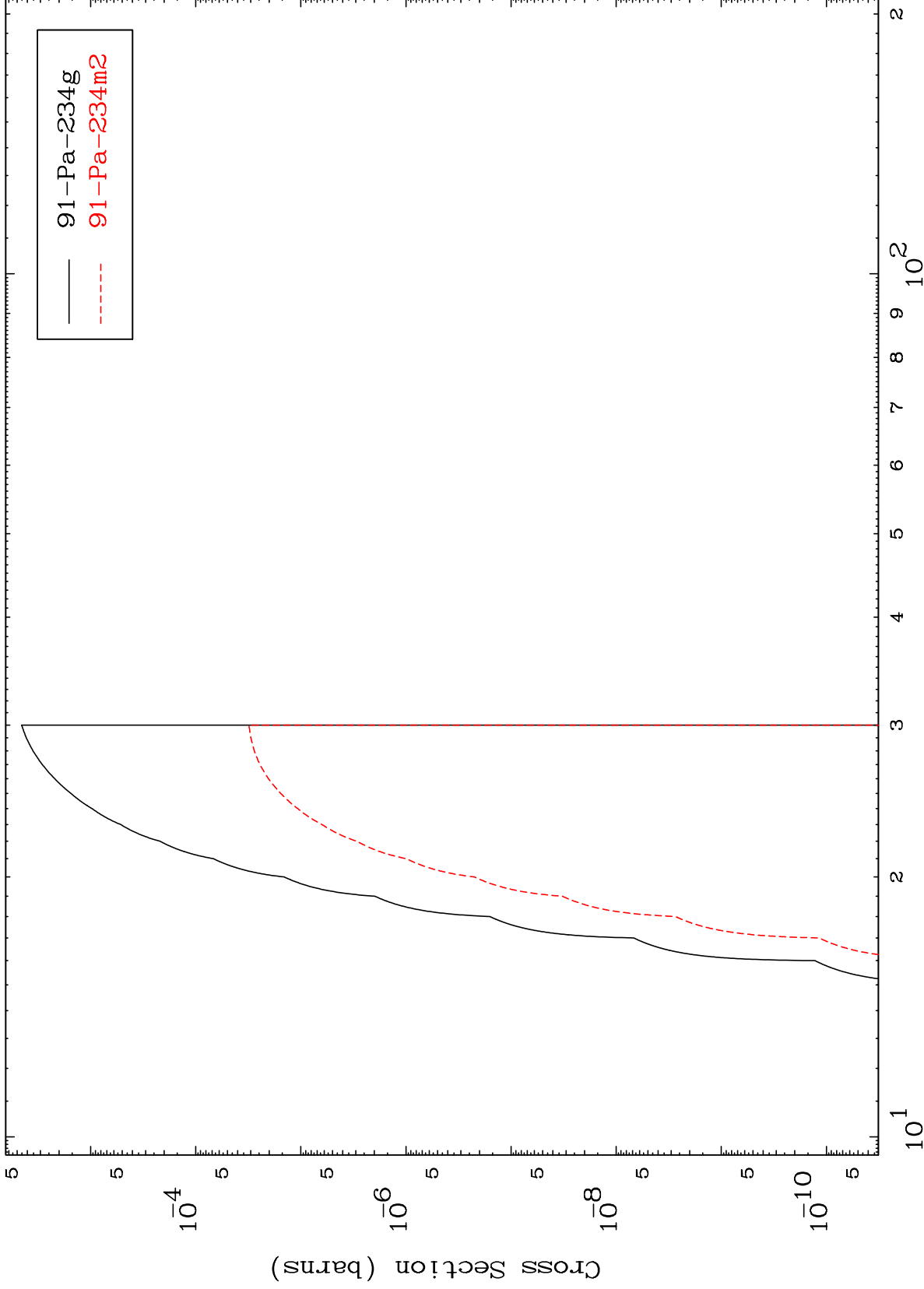
92-U -237

MAT 9234

(n,n') t

92-U -237

Radionuclide Production Cross Section



26

Incident Energy (MeV)

92-U -237

MAT 92334

(n,3n) p

92-U -237

Radionuclide Production Cross Section

