

Program EVALPLOT  
(Version 2021-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail: [redcullen1@comcast.net](mailto:redcullen1@comcast.net)

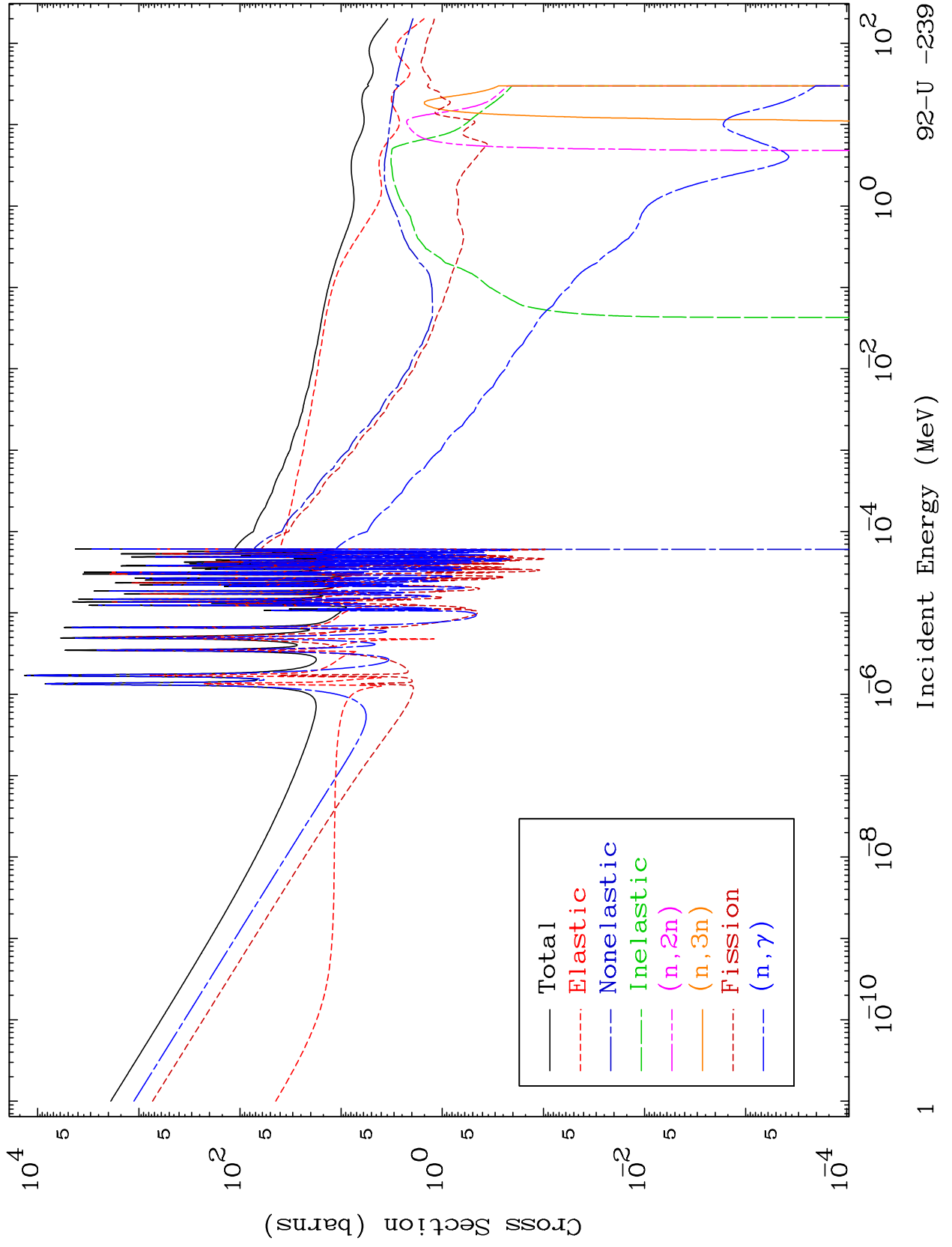
Web: [redcullen1.net/HOMEPAGE.NEW](http://redcullen1.net/HOMEPAGE.NEW)

Press Mouse Button to Start

MAT 9240

Neutron Major  
293 Kelvin Cross Sections

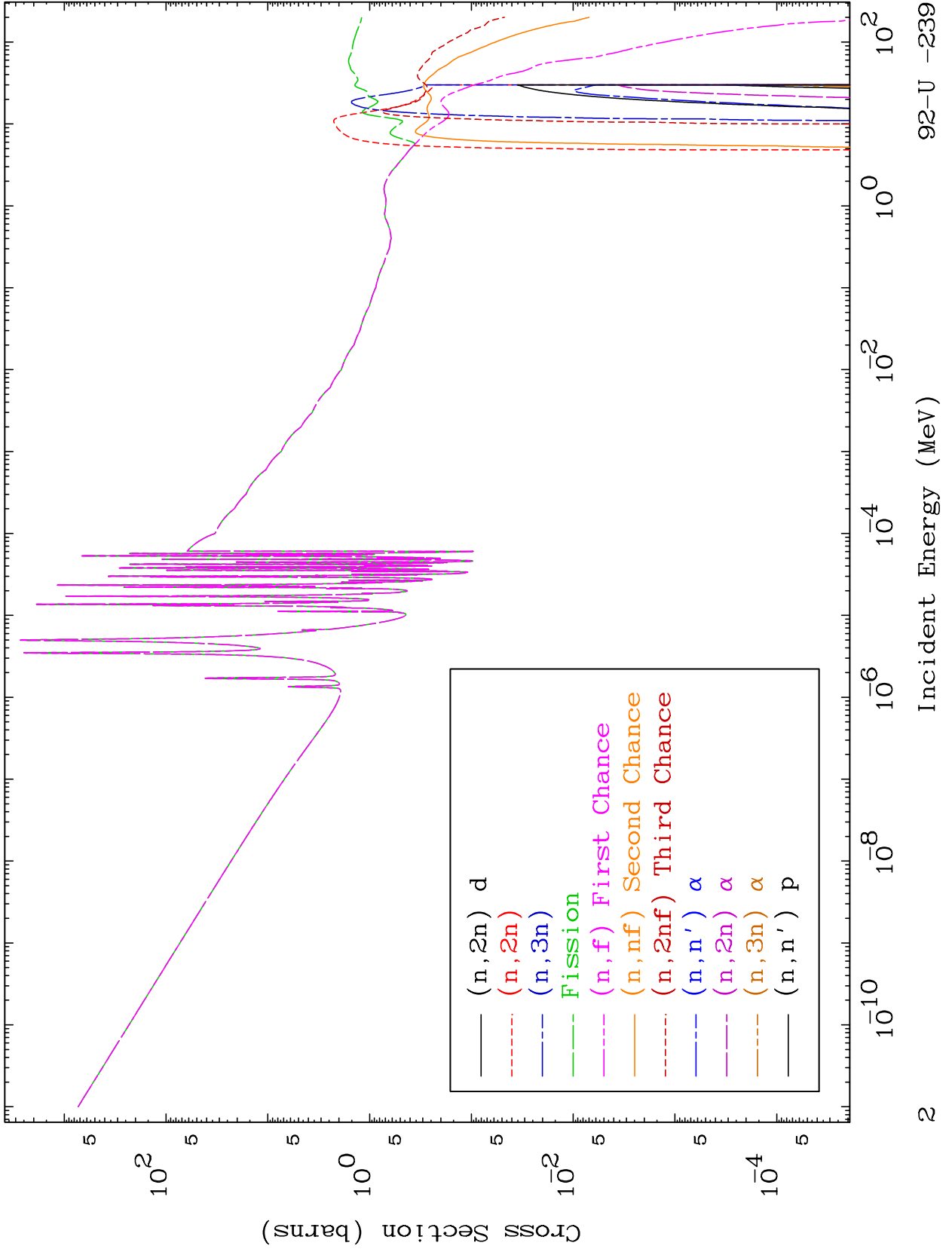
92-U -239



MAT 9240

Neutron Absorption  
293 Kelvin Cross Sections

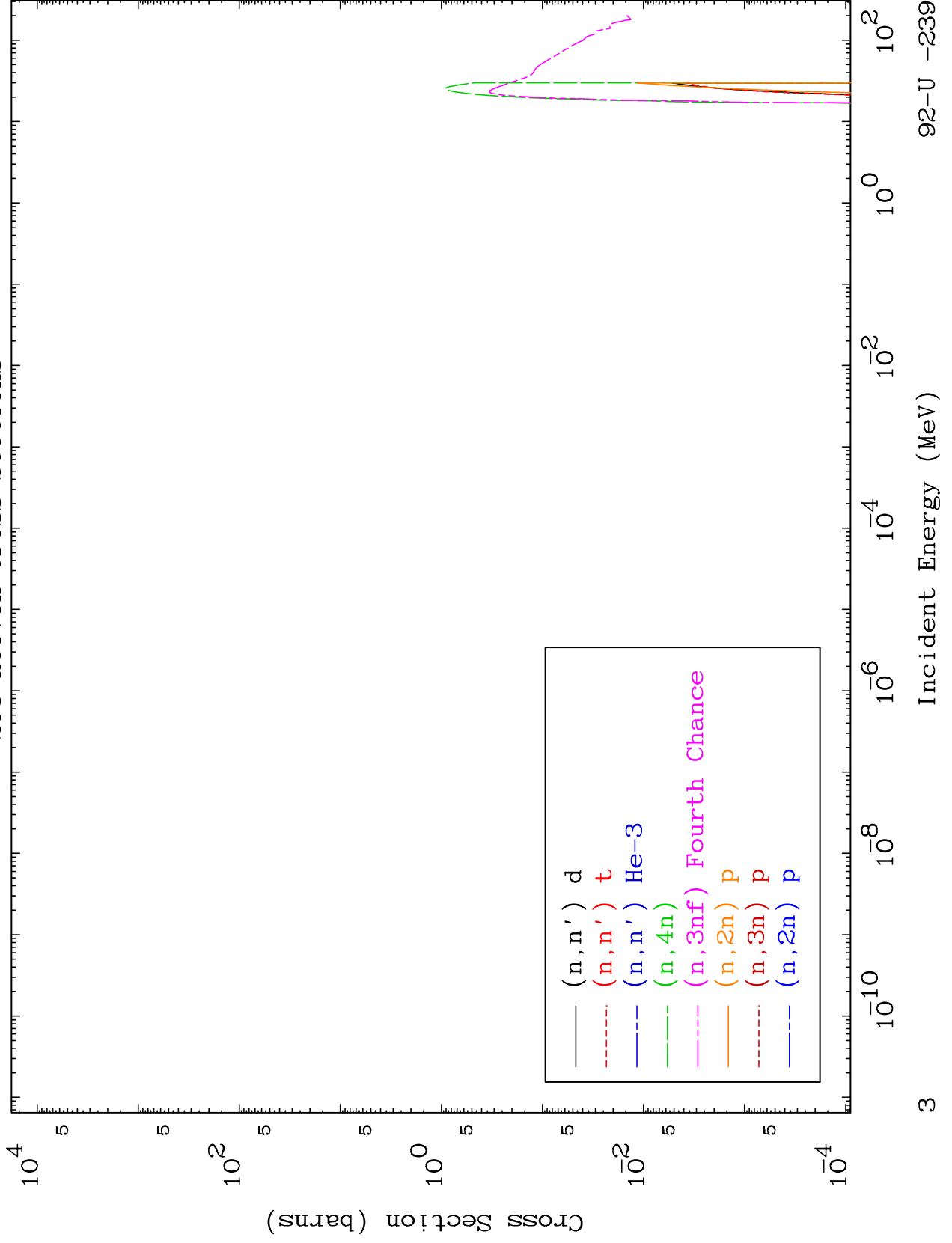
92-U -239



MAT 9240

Neutron Absorption  
293 Kelvin Cross Sections

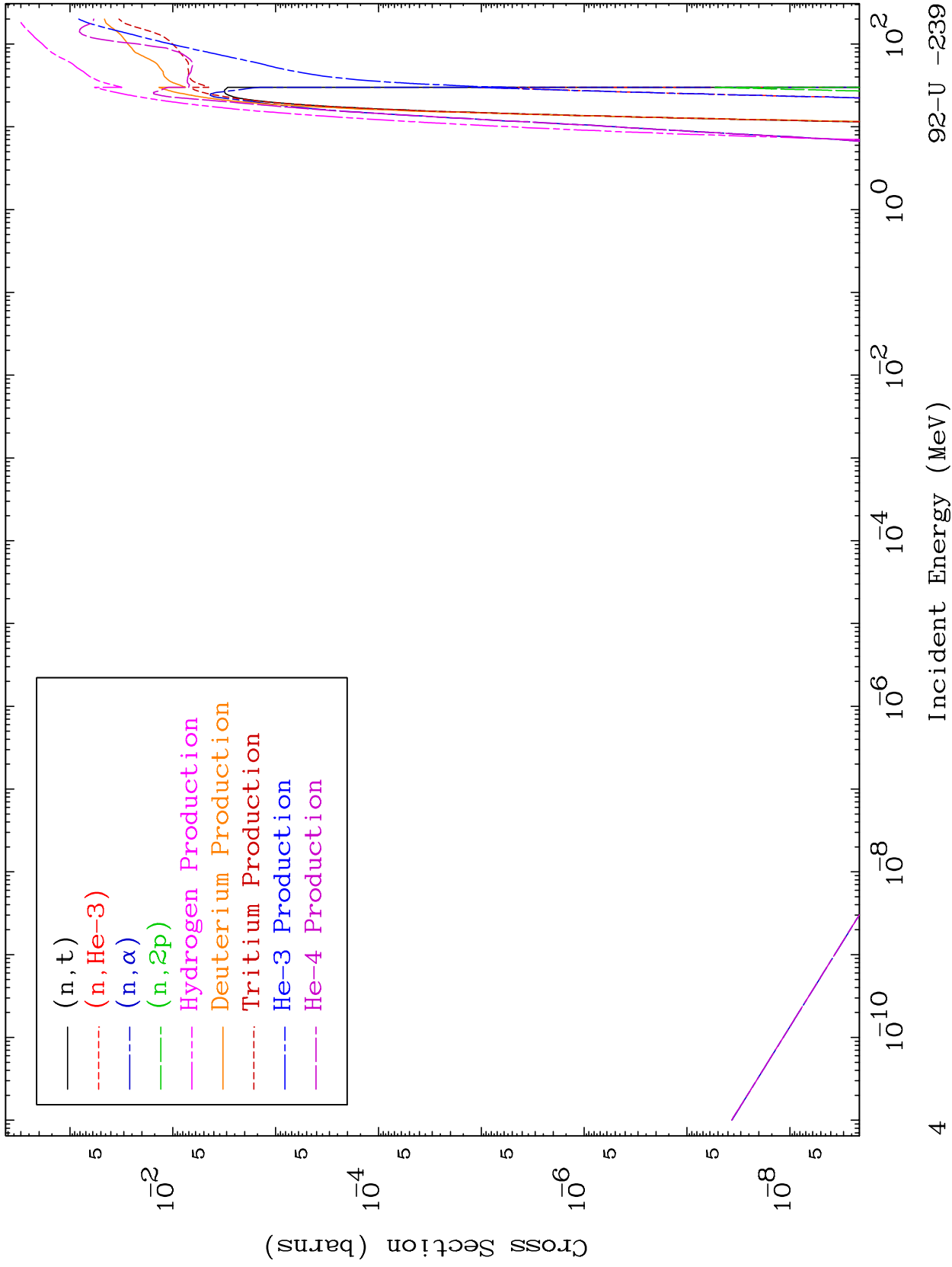
92-U -239



MAT 9240

Neutron Absorption  
293 Kelvin Cross Sections

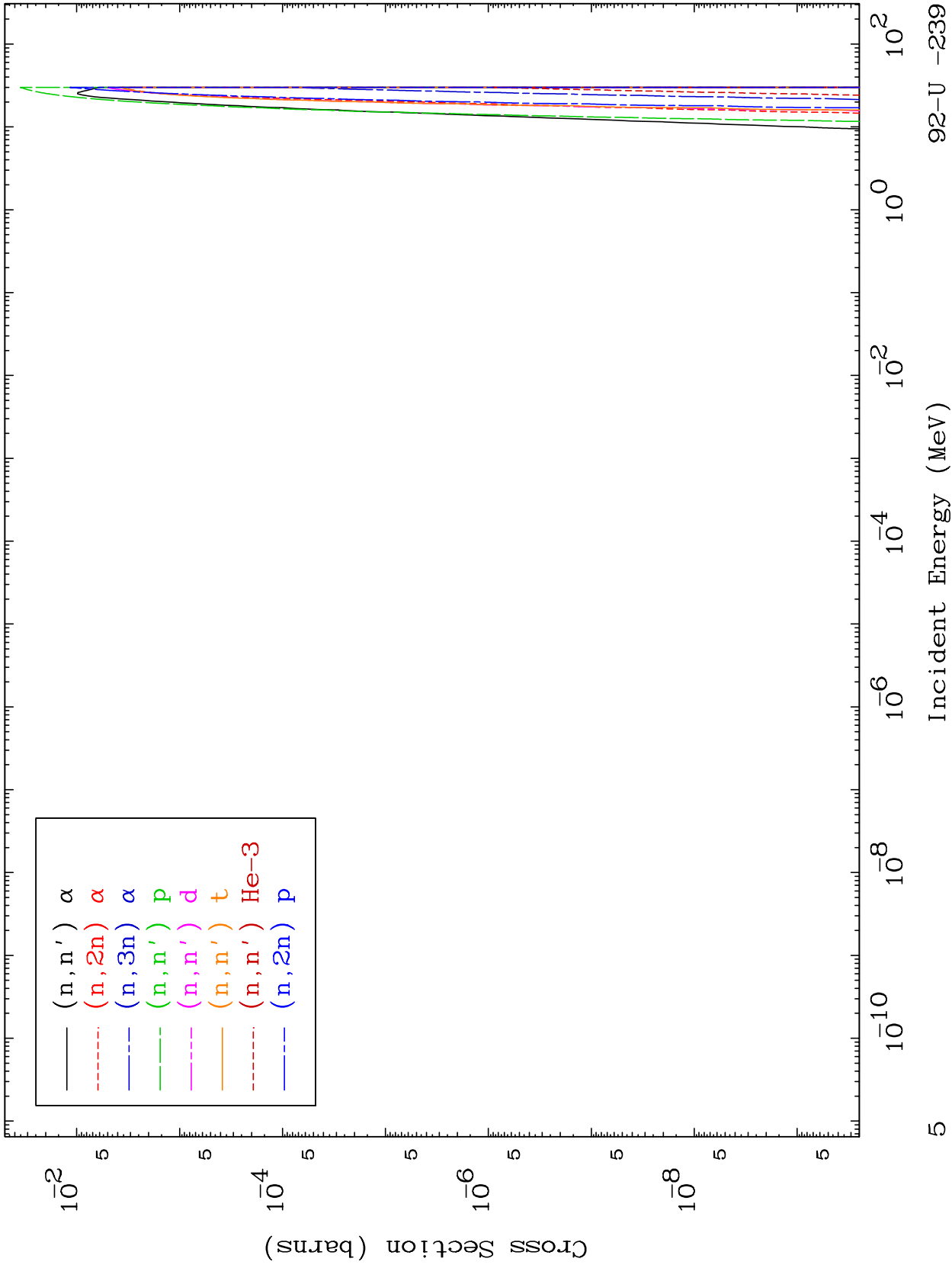
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Charged Particle  
293 Kelvin Cross Sections

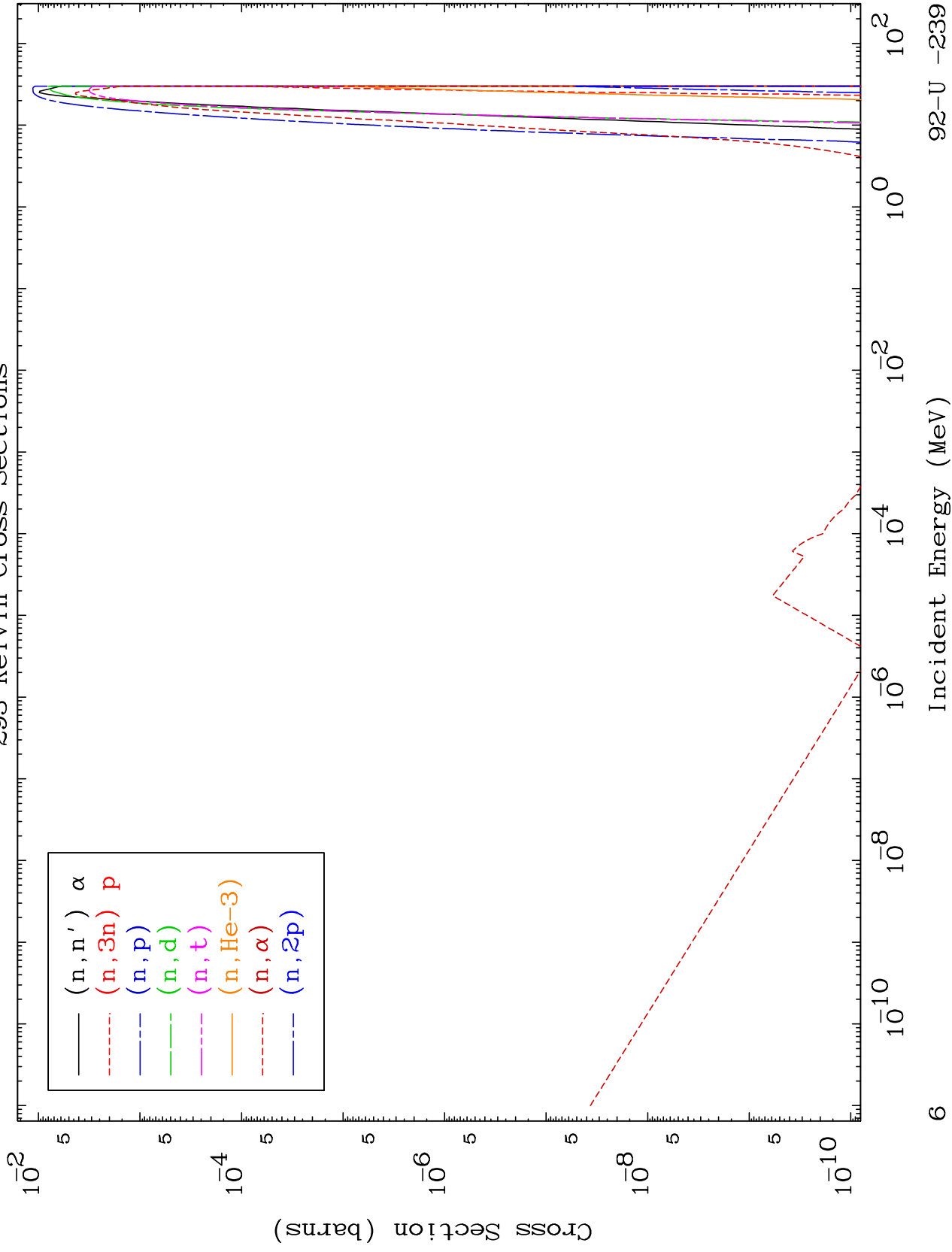
92-U -239



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Charged Particle  
293 Kelvin Cross Sections

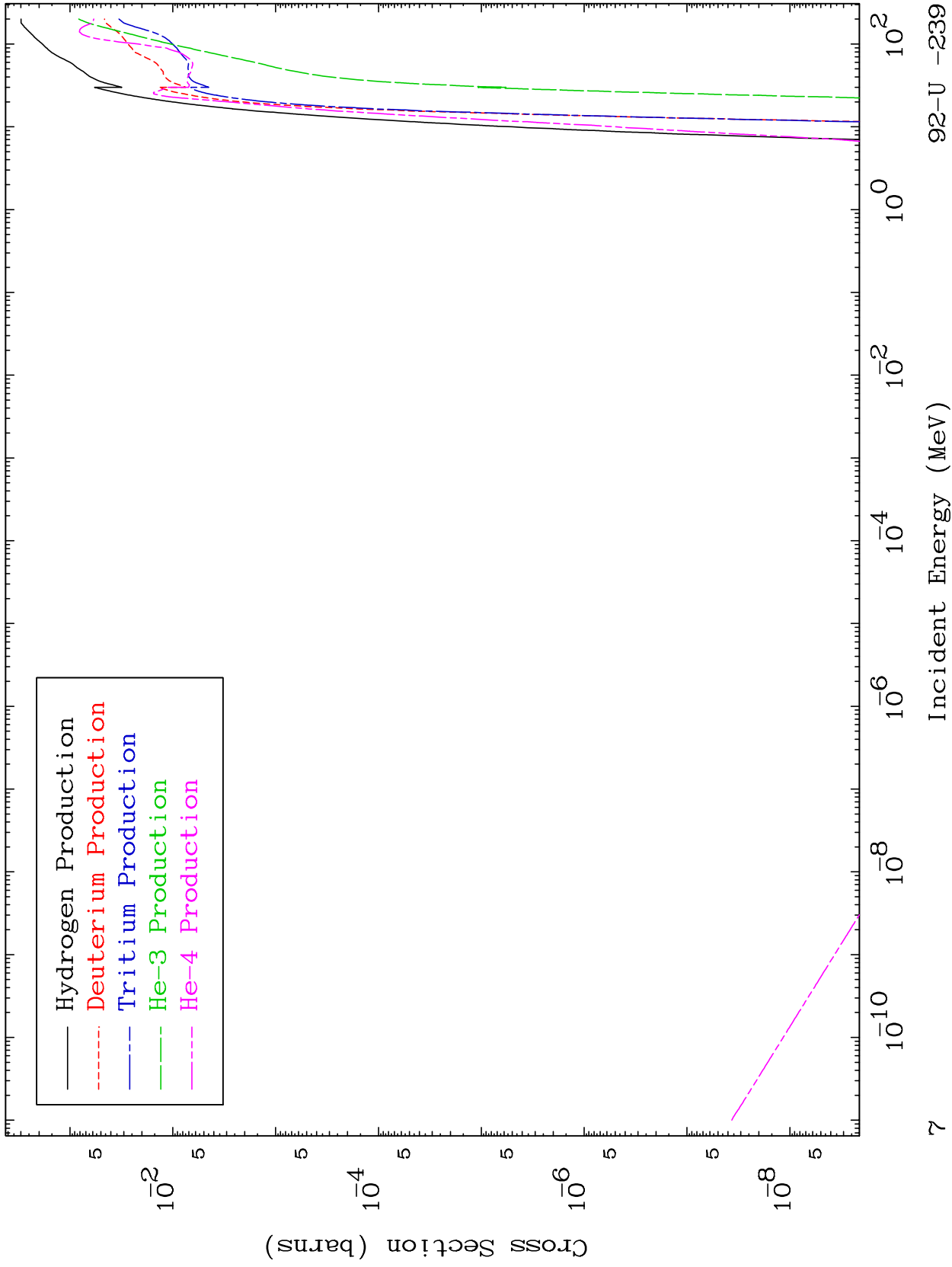
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Particle Production  
293 Kelvin Cross Sections

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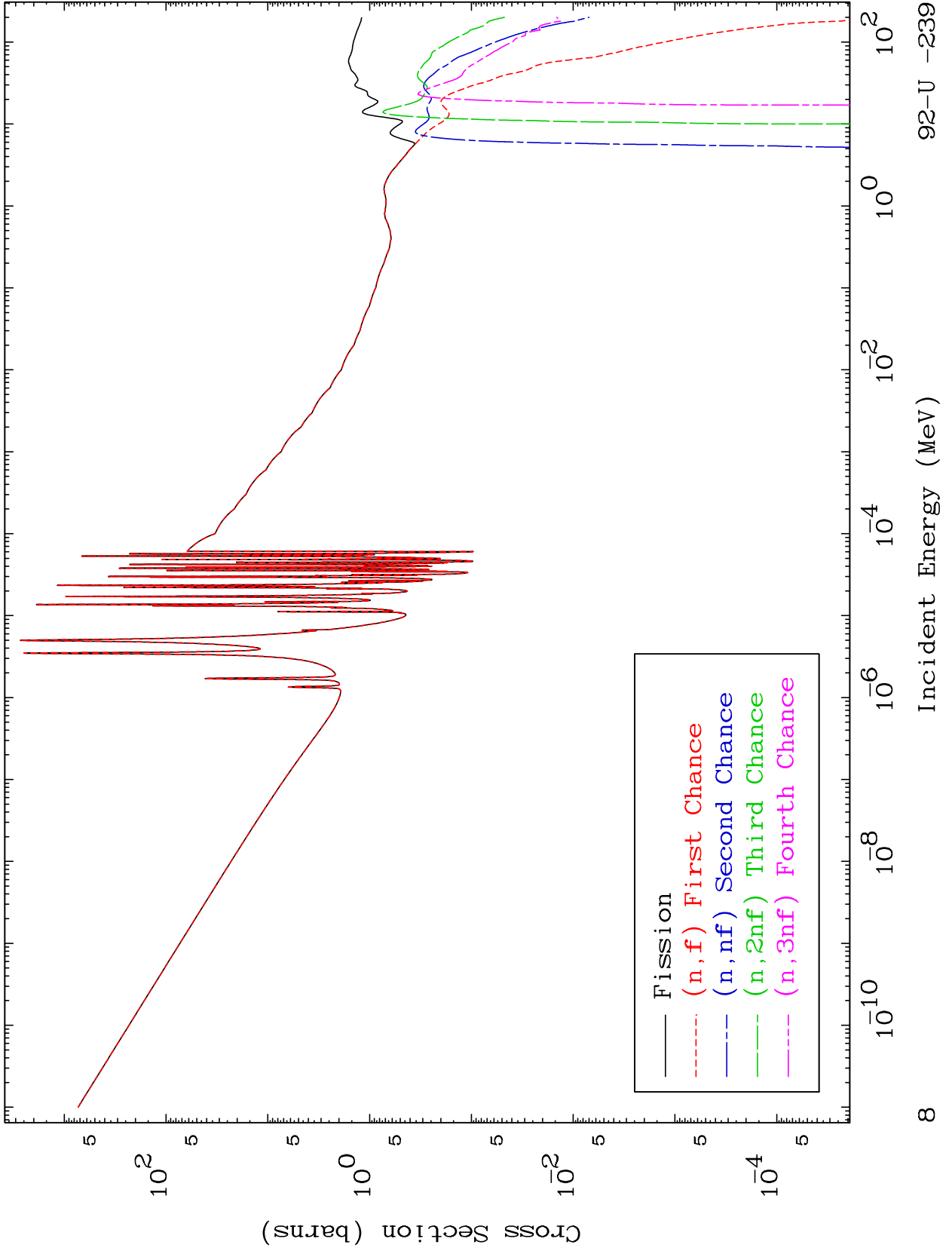


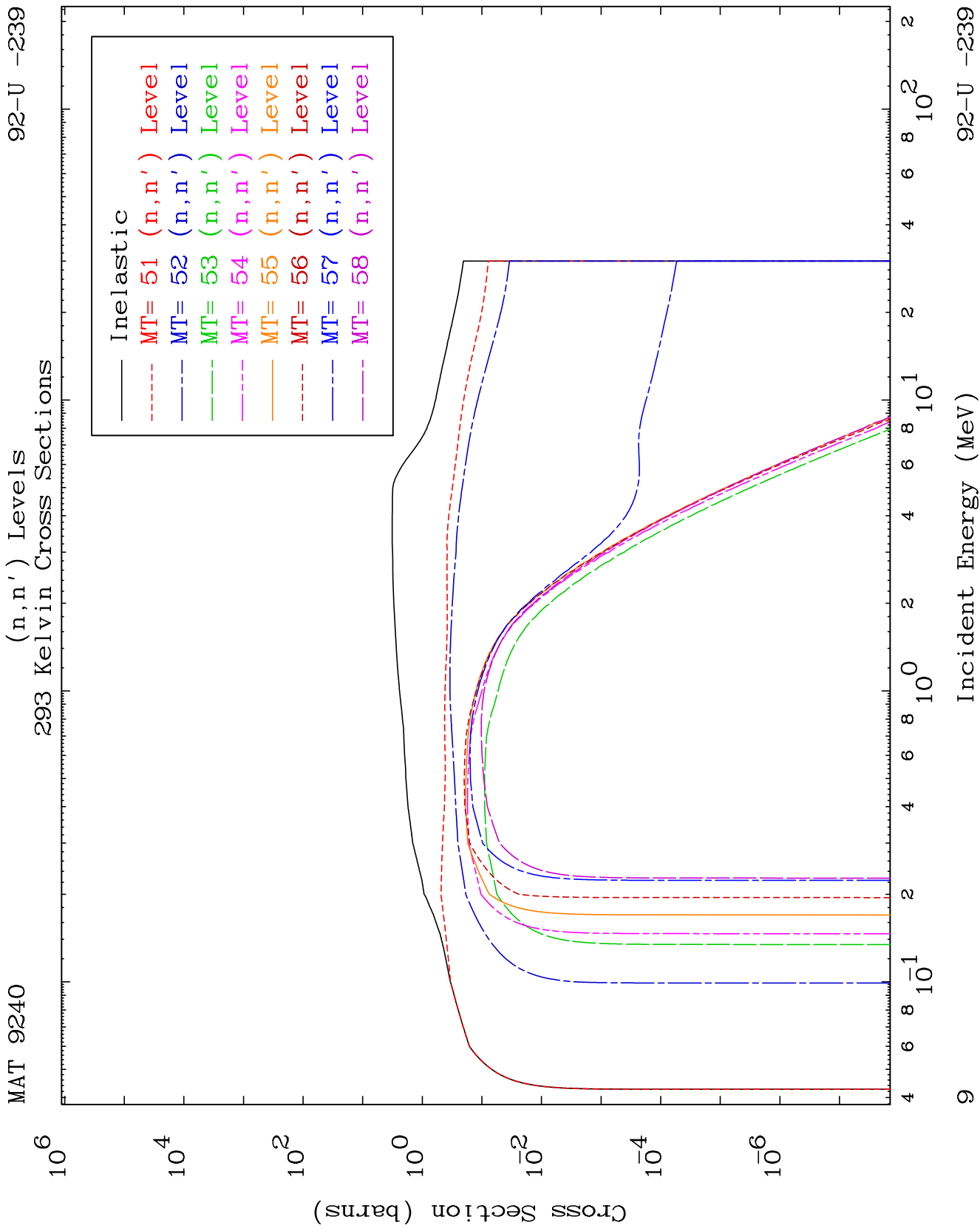


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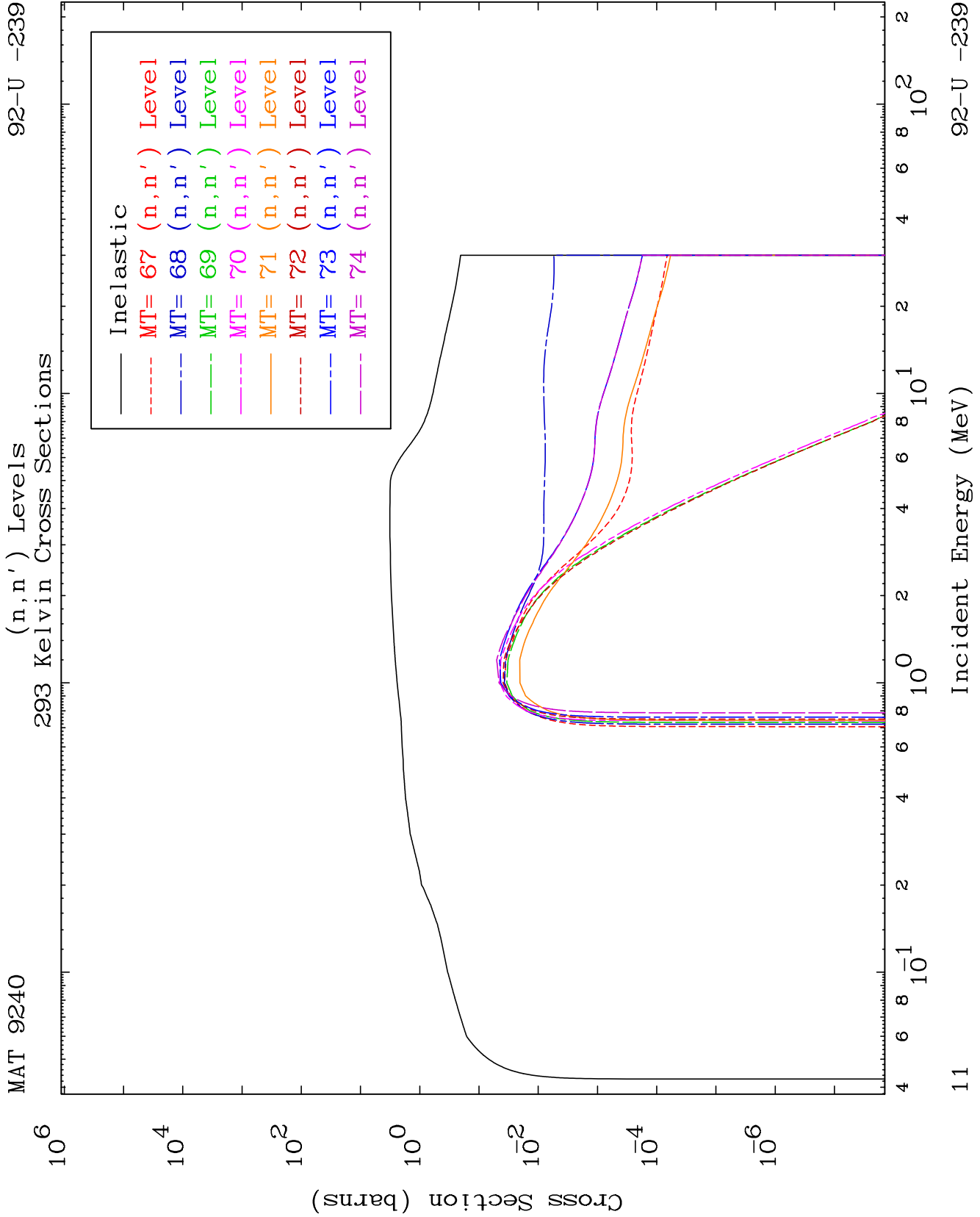
Fission  
293 Kelvin Cross Sections

92-U -239





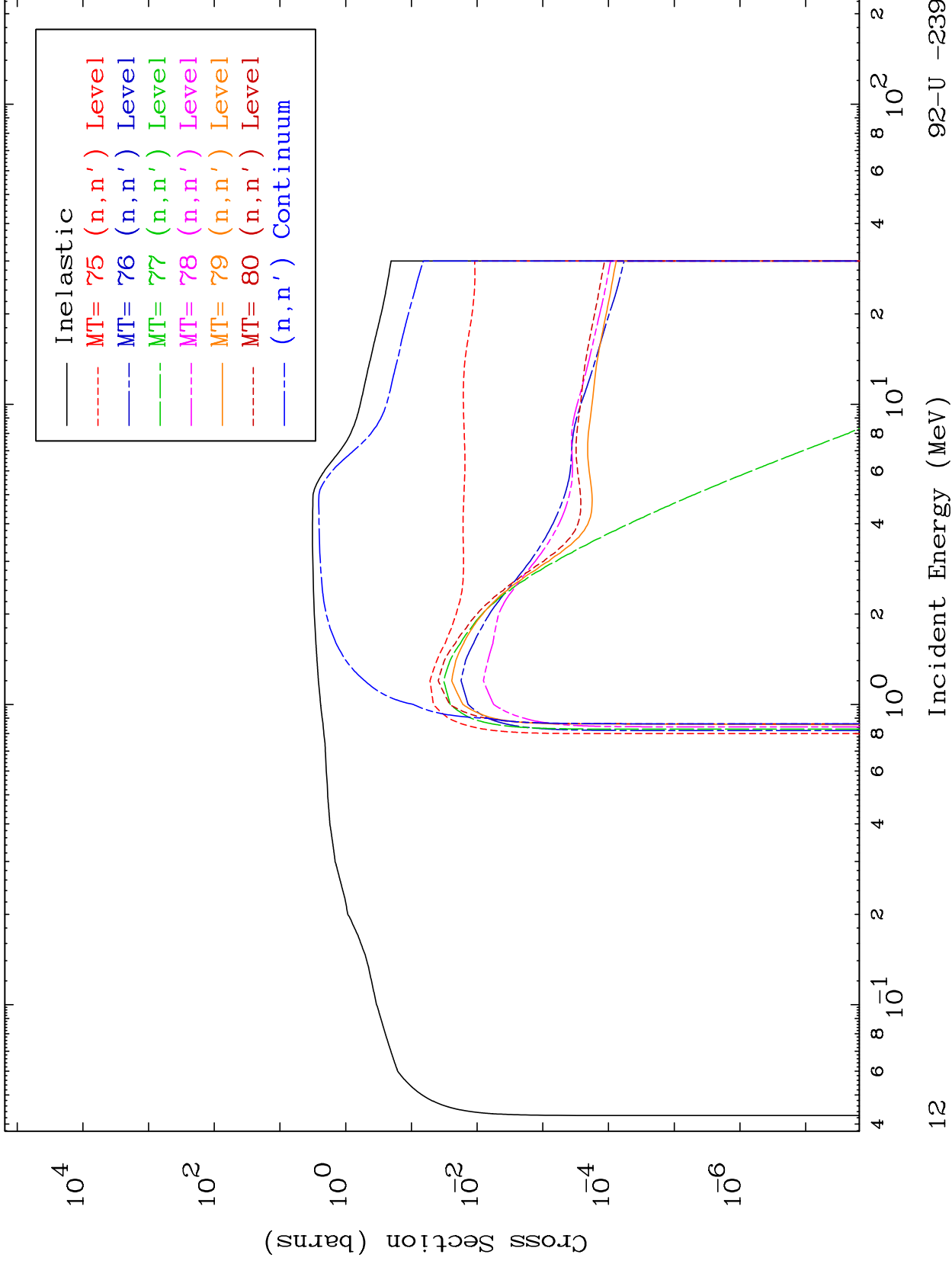




MAT 9240

(n,n') Levels  
293 Kelvin Cross Sections

92-U -239



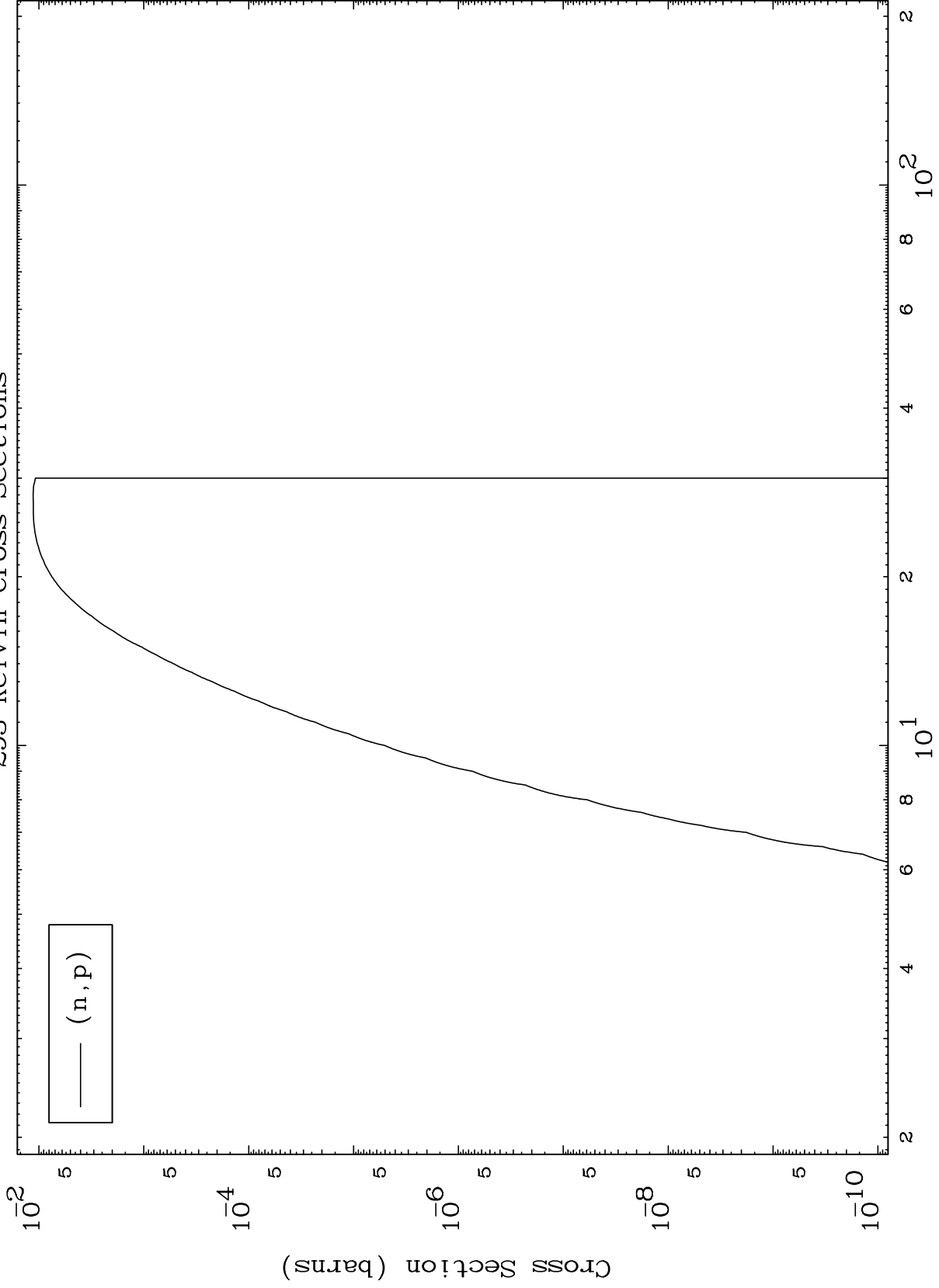
12

92-U -239

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(n,p) Levels  
293 Kelvin Cross Sections

92-U -239



13

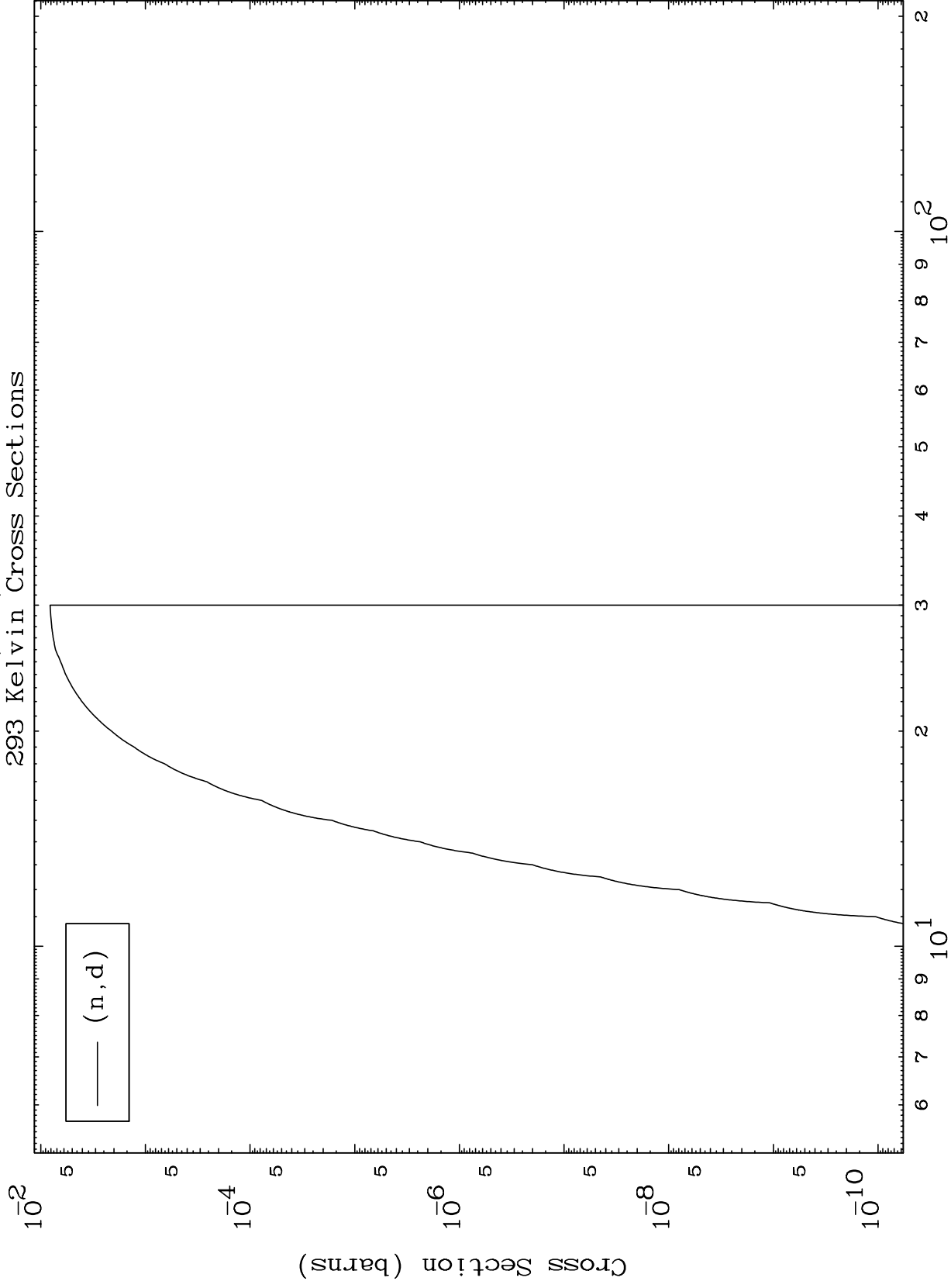
Incident Energy (MeV)

92-U -239

MAT 9240

(n,d) Levels  
293 Kelvin Cross Sections

92-U -239



14

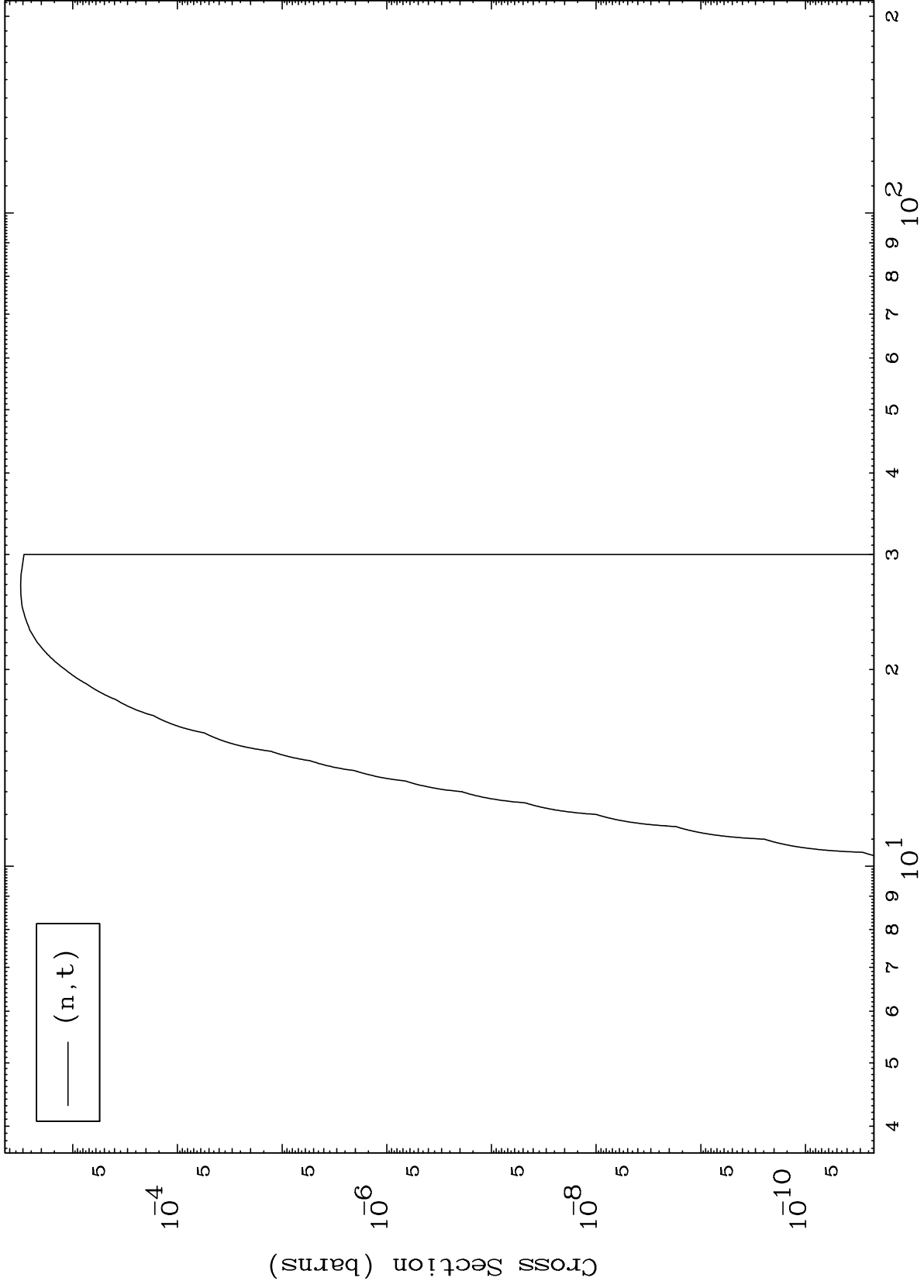
Incident Energy (MeV)

92-U -239

MAT 9240

(n,t) Levels  
293 Kelvin Cross Sections

92-U -239



15

Incident Energy (MeV)

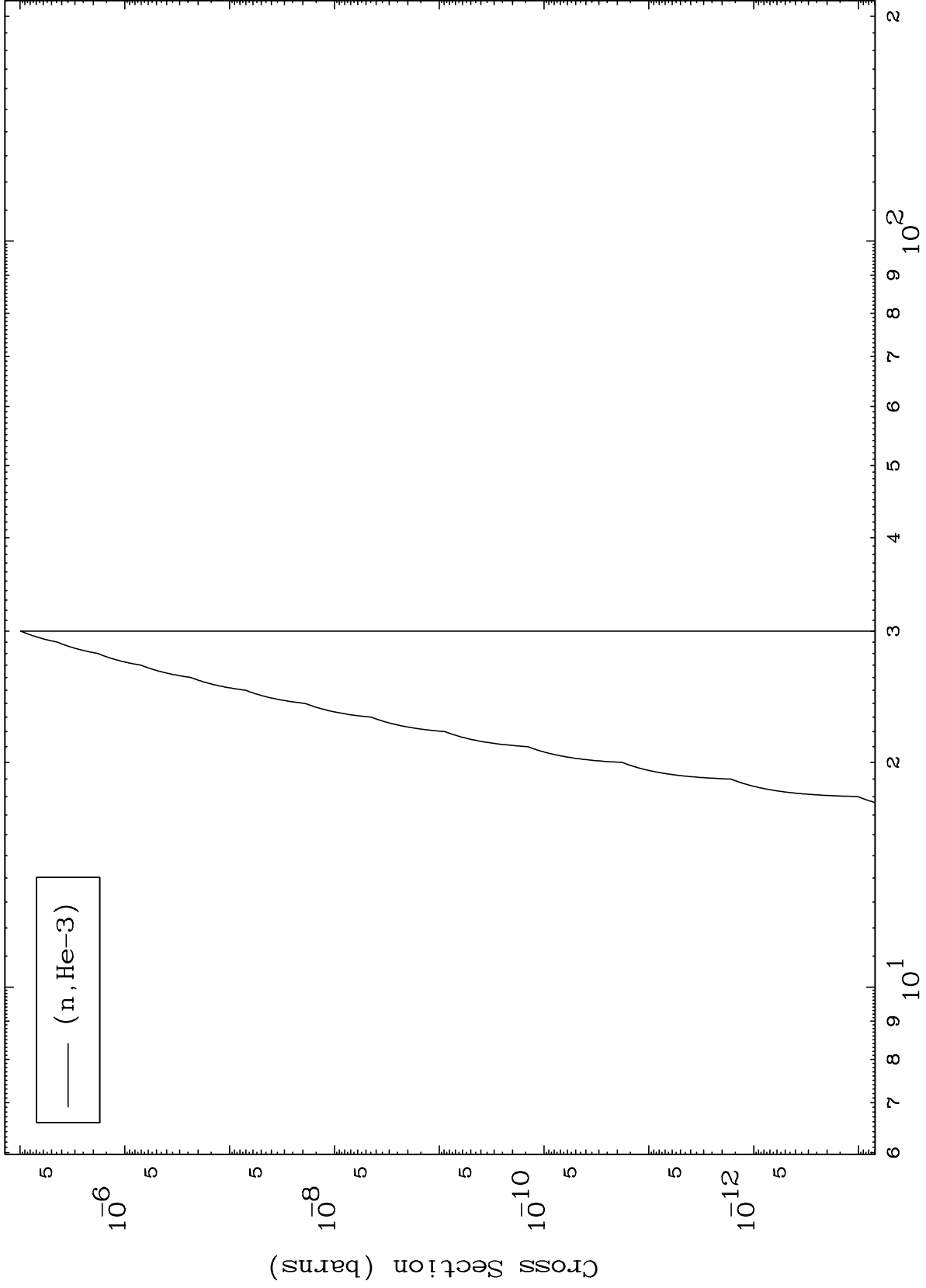
92-U -239



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(n,He3) Levels  
293 Kelvin Cross Sections

92-U -239



16

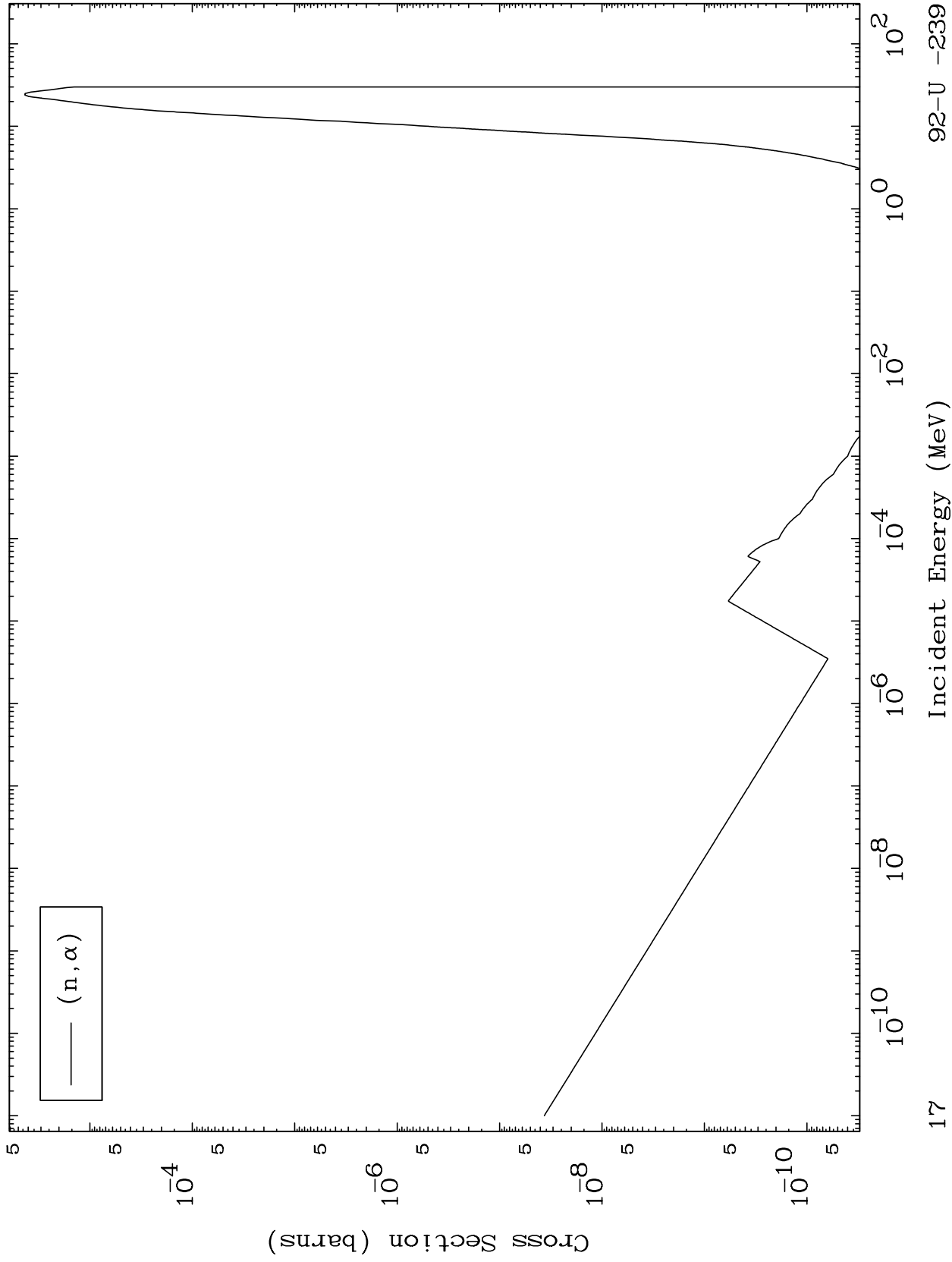
Incident Energy (MeV)

92-U -239

MAT 9240

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

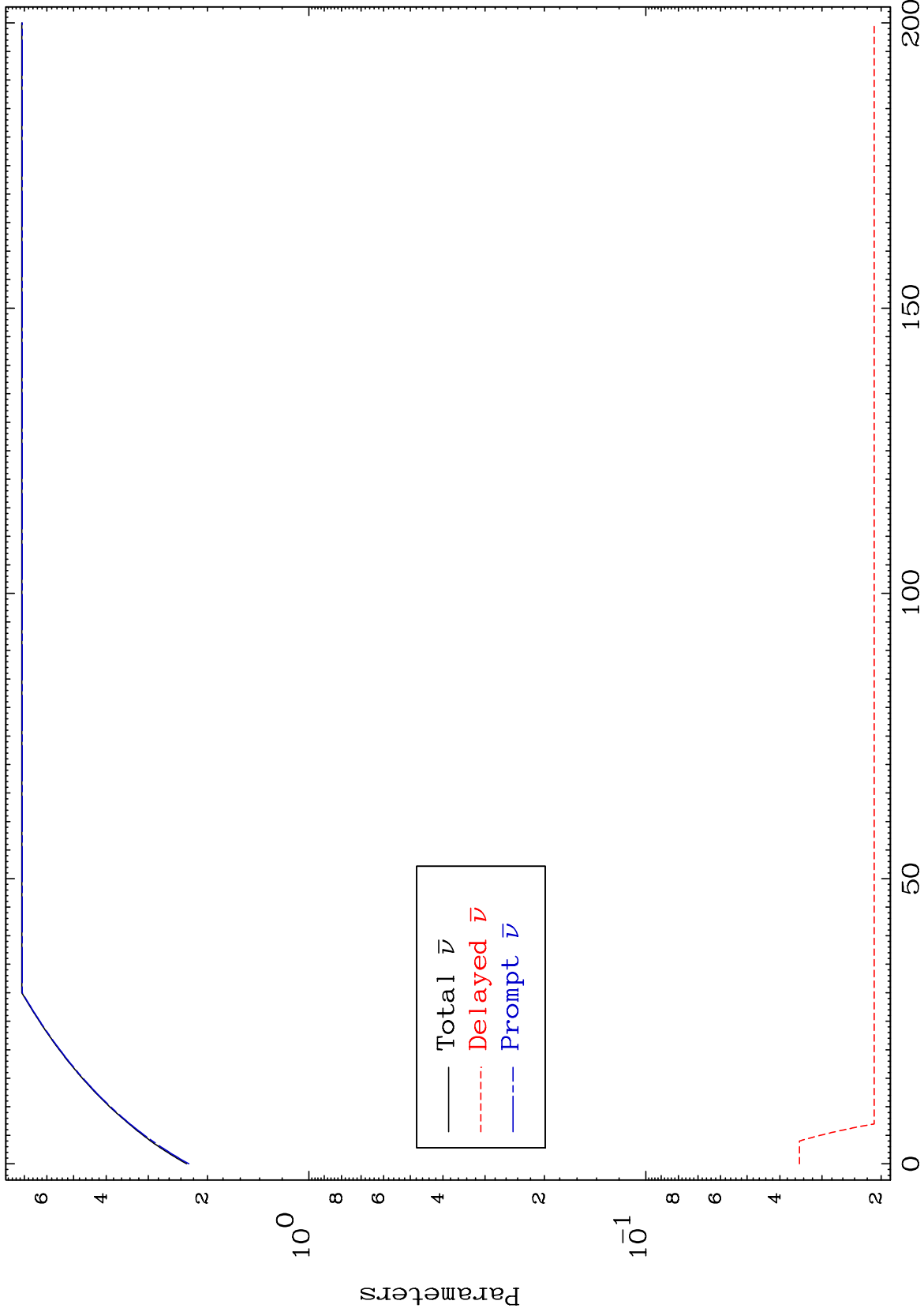
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Energy Release  
Parameters

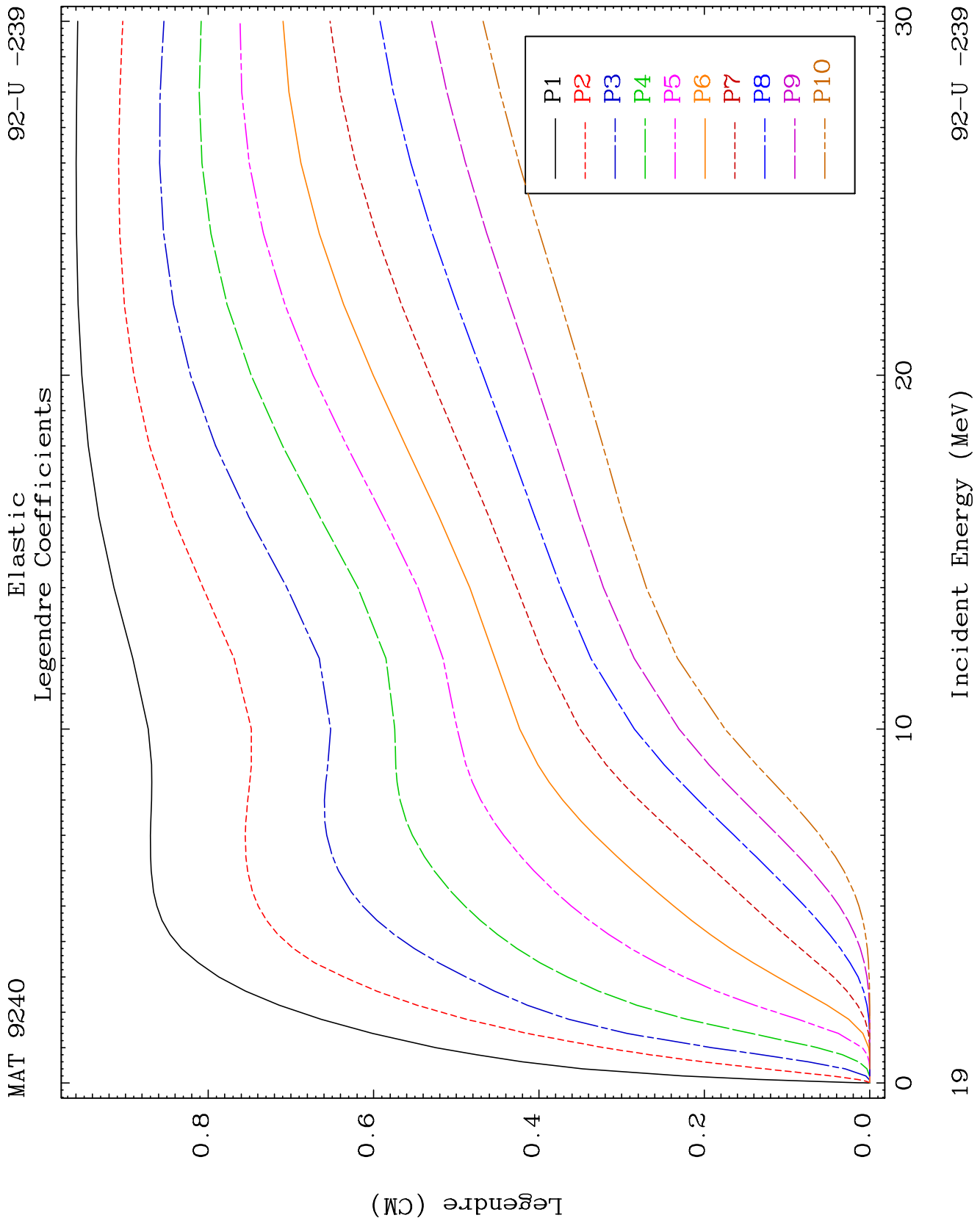
92-U -239

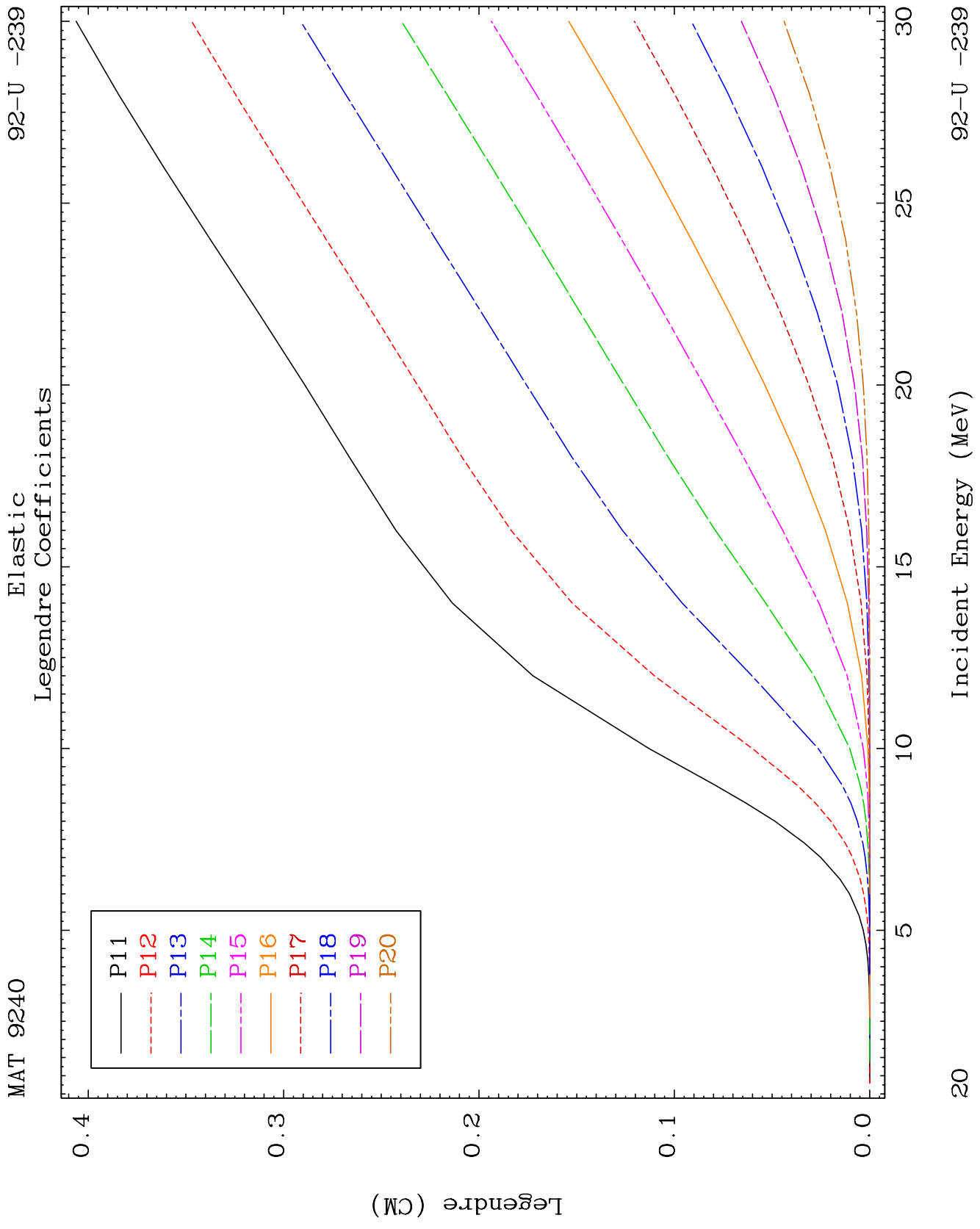


18

Incident Energy (MeV)

92-U -239

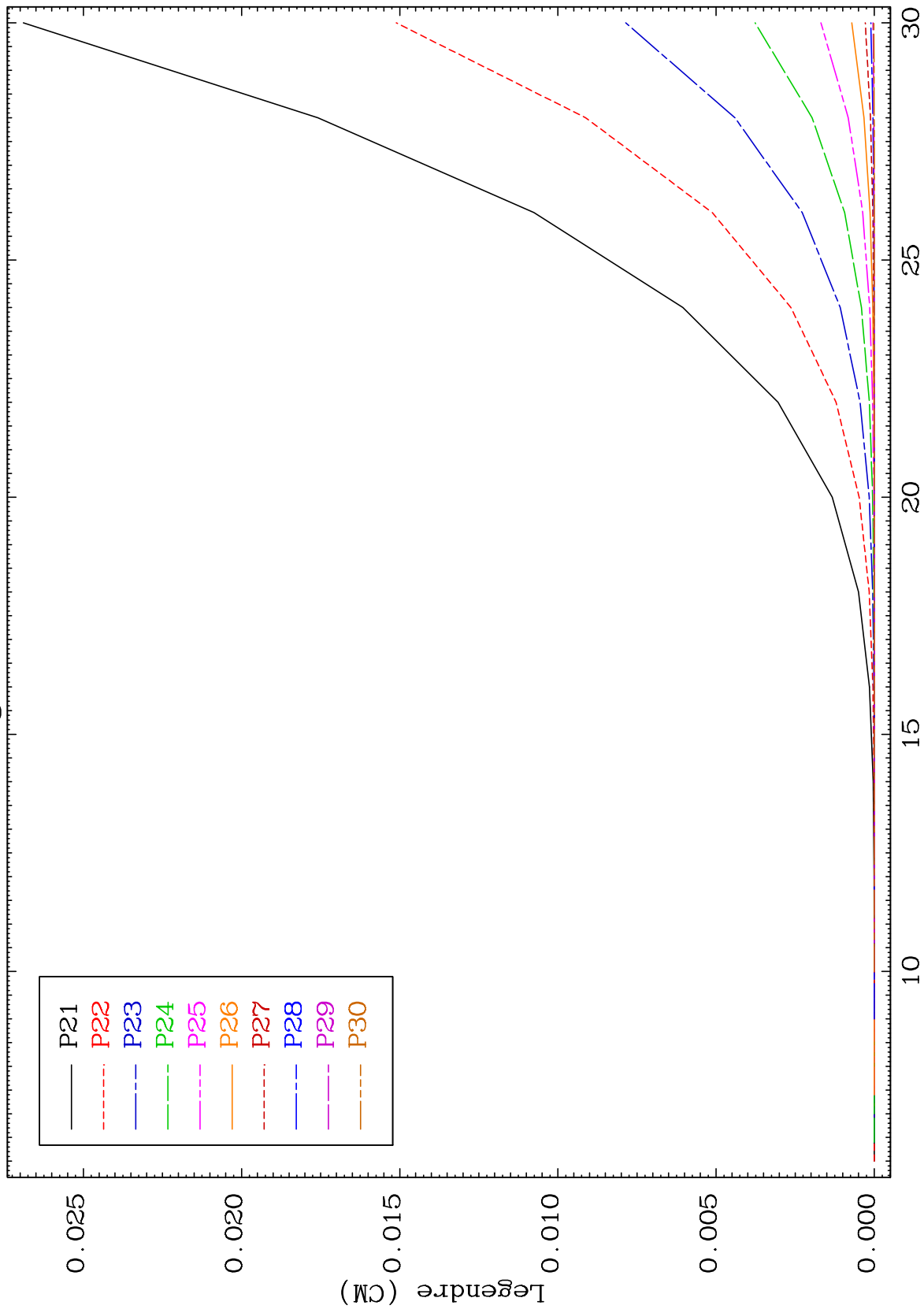




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### Elastic Legendre Coefficients

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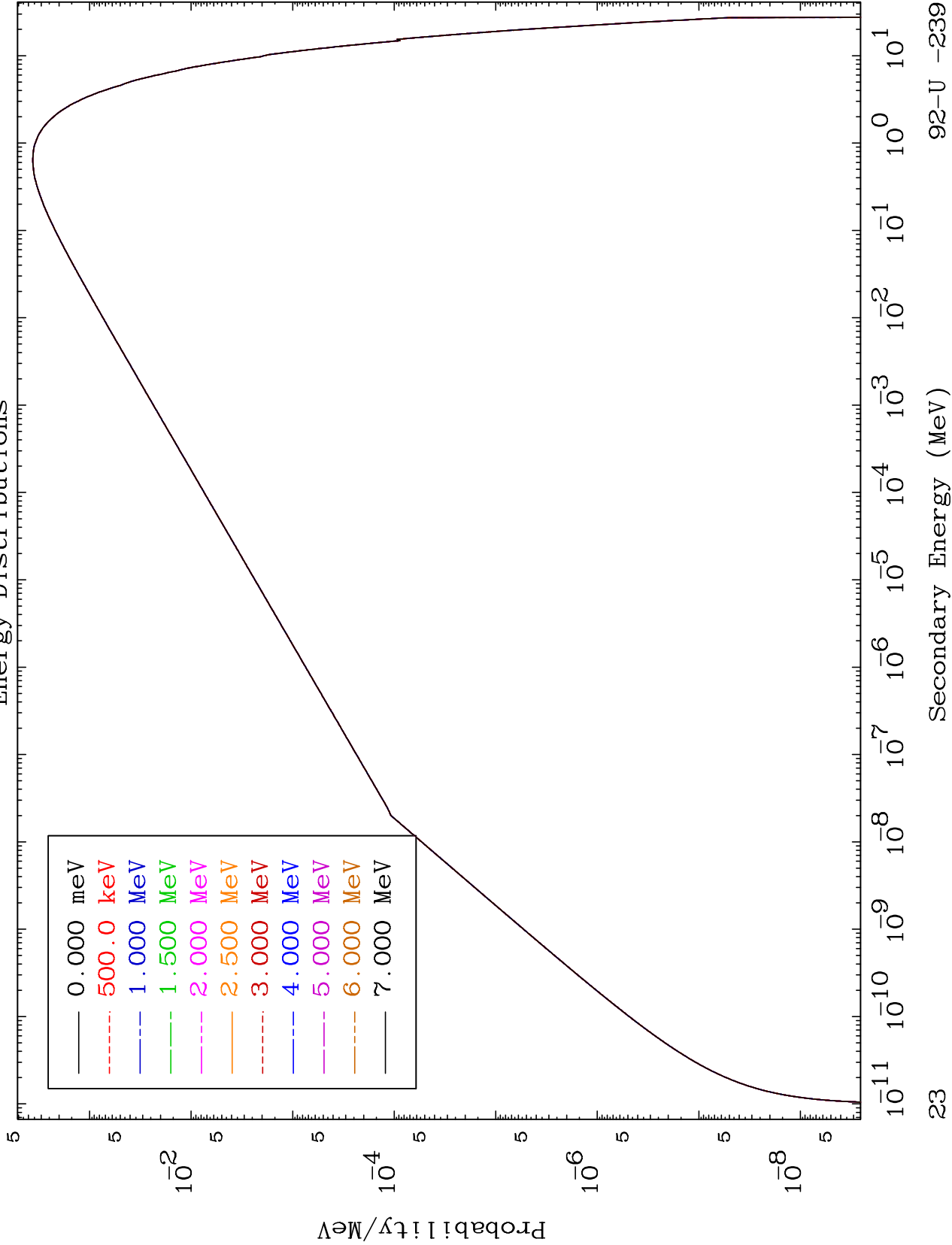
21

Incident Energy (MeV)

92-U -239



Fission Energy Distributions

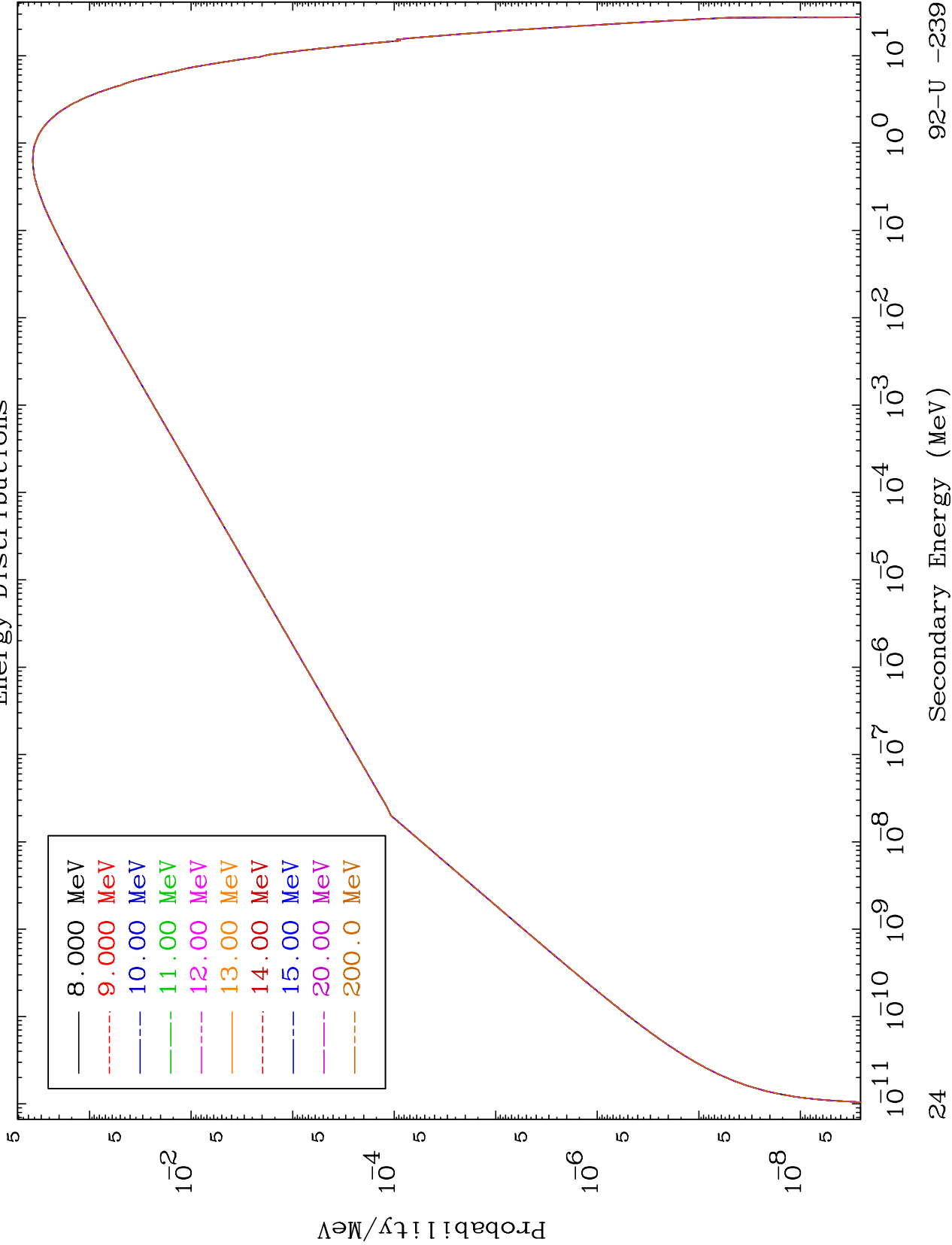




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### Fission Energy Distributions

92-U -239



24

Secondary Energy (MeV)

92-U -239